15/4-1

Nuclear Regulatory Commission

OPERATING UNITS STATUS REPORT

DATA AS OF 12-31-79

NUREG 0020
VOL. 4 NO. 1
JANUARY 1980

NEWARK PUBL'C LIBRARY
MAR 26 1980
U. S. GOVT. PUBS.
DEPOSITORY 376

LICENSED OPERATING REACTORS

Y3.N88:

Available from
GPO Sales Program
Division of Technical Information and Document Control
U. S. Nuclear Regulatory Commission
Washington, D.C. 20555
and
National Technical Information Service
Springfield, Virginia 22161
703-326-8543
A Year's Subscription, 12 Issues, for this Publication is
$120.00 North American Continent ($240 00 Other).
Multiple Subscriptions Entered at the Same Time to the Same Address
Cost $90.00 North American Continent ($180.00 Other) for each Additional Subscription.

Single copies are $12.50 North American Continent ($25.00 Other).

This document is provided free of charge to those who participate in the data collection program, and to the Nuclear Regulatory Commission's local public document rooms.

Issued by the U.S. Nuclear Regulatory Commission

PUBLISHED: 02/28/80

Nuclear Regulatory Commission

OPERATING UNITS STATUS REPORT

DATA AS OF 12-31-79

NUREG 0020
VOL. 4 NO. 1
JANUARY 1980

LICENSED OPERATING REACTORS

READEX FICHE

AUTHORIZATION AND CLEARANCE

THE U.S. NUCLEAR REGULATORY COMMISSION'S OFFICE OF MANAGEMENT AND PROGRAM ANALYSIS PUBLISHES THIS MONTHLY STATUS REPORT "AS PART OF THE REPORTING REQUIREMENTS IN SECTION 50.36 OF 10 CFR PART 50 UNDER GAO CLEARANCE NUMBER B-180225, WITH AN EXPIRATION DATE OF SEPTEMBER 30, 1981," AS STATED IN THE OCTOBER 3, 1978 LETTER FROM JOHN M. LOVELADY, ASSISTANT DIRECTOR, GENERAL GOVERNMENT DIVISION, U. S. GENERAL ACCOUNTING OFFICE, TO J.M. FELTON, DIRECTOR, DIVISION OF RULES AND RECORDS, U.S. NUCLEAR REGULATORY COMMISSION.

STATEMENT OF PURPOSE

The U. S. Nuclear Regulatory Commission's monthly OPERATING UNITS STATUS REPORT – LICENSED OPERATING REACTORS, provides data on the operation of nuclear units as timely and accurately as possible. This information is collected by the Office of Management and Program Analysis from the Headquarters Staff of NRC's Office of Inspection and Enforcement, from NRC's Regional Offices, and from utilities. Since all of the data concerning operation of the units is provided by the utility operators less than two weeks after the end of the month, necessary corrections to published information are shown on the ERRATA page.

This report is divided into three sections: the first contains monthly highlights and statistics for commercial operating units, and errata from previously reported data; the second is a compilation of detailed information on each unit, provided by NRC Regional Offices and IE Headquarters; and the third section is an appendix containing nuclear availability and capacity factors, forced outage rate statistics, spent fuel storage capability, reactor-years of experience and non-power reactors in the United States.

Throughout the report, statistical factors for periods greater than a report month, or for more than one unit, are computed as the arithmetic average of each unit's individual operating statistics. The statistical factors for each unit for the report month are computed from actual power production for the month. The factors for the life-span of each unit (the "Cumulative" column) are reported by the utility and are not entirely re-computed by NRC. Utility power production data is checked for consistency with previously submitted statistics.

It is hoped this status report proves informative and helpful to all agencies and individuals interested in analyzing trends in the nuclear industry which might have safety implications, or in maintaining an awareness of the U. S. energy situation as a whole.

PHILIP A. ROSS
REVIEW COORDINATOR

DEBBIE NICHOLS
TECHNICAL ASSISTANT

TABLE OF CONTENTS

GLOSSARY ii
INDEX TO LICENSED UNITS iii

SECTION 1 – CURRENT DATA SUMMARIES

MONTHLY HIGHLIGHTS OF COMMERCIAL NUCLEAR POWER UNITS 1-2

- Licensed Power Reactors 1-2
- Power Generation 1-2
- Actual vs. Potential Energy Production 1-2
- Outage Data 1-2
- Derated Units 1-2
- Shutdowns - Number and Reasons 1-3
- Unit Availability, Capacity, and Forced Outage Rate 1-3
- Refueling Schedule 1-3

AVERAGE CAPACITY FACTORS BY VENDOR 1-4

AVERAGE DAILY POWER LEVEL FOR ALL COMMERCIAL OPERATING UNITS 1-5

ERRATA 1-7

SECTION 2 – OPERATING POWER REACTORS

ARKANSAS 1 THROUGH ZION 2 PAGES A–2 TO Z–5

- Facility Data
- Inspection and Enforcement Status
- Reports Received from Licensee
- Operating Status
- Average Daily Power Level (MWe)
- Unit Shutdowns/Reductions

SECTION 3 – APPENDIX

NUCLEAR POWER GENERATION (Availability & Capacity Factors), 1974 to PRESENT 3-2
NUCLEAR POWER GENERATION (Availability Factor, Capacity Factor & Forced Outage Rate), 1975 to PRESENT 3-3
STATUS OF SPENT FUEL STORAGE CAPABILITY 3-4
REACTOR-YEARS OF EXPERIENCE 3-5
NON-POWER REACTORS IN THE U.S. 3-6

GLOSSARY

AVERAGE DAILY POWER LEVEL (MWe)	The net electrical energy generated during the day (measured from 0001 to 2400 hours inclusive) in megawatts hours divided by 24 hours.
LICENSED THERMAL POWER (MWt)	The maximum thermal power of the reactor authorized by the NRC, expressed in megawatts.
DATE OF COMMERCIAL OPERATION	Date unit was declared by utility owner to be available for the regular production of electricity; usually related to satisfactory completion of qualification tests as specified in the purchase contract and to accounting policies and practices of utility.
DESIGN ELECTRICAL RATING (DER) (Net MWe)	The nominal net electrical output of the unit specified by the utility and used for the purpose of plant design.
FORCED OUTAGE	An outage required to be initiated no later than the weekend following discovery of an off-normal condition.
FORCED OUTAGE HOURS	The clock hours during the report period that a unit is unavailable due to forced outages.
GROSS ELECTRICAL ENERGY GENERATED (MWH)	Electrical output of the unit during the report period as measured at the output terminals of the turbine generator, in megawatts hours.
GROSS HOURS	The clock hours from the beginning of a specified situation until its end. For outage durations, the clock hours during which the unit is not in power production.
GROSS THERMAL ENERGY GENERATED (MWH)	The thermal energy produced by the unit during the report period as measured or computed by the licensee in megawatt hours.
HOURS GENERATOR ON-LINE	Also, "Unit Service Hours." The total clock hours in the report period during which the unit operated with breakers closed to the station bus. These hours added to the total outage hours experienced by the unit during the report period, shall equal the hours in the report period.
HOURS IN REPORTING PERIOD	For units in power ascension at the end of the period, the gross hours from the beginning of the period or the first electrical production, whichever comes last, to the end of the period. For units in commercial operation at the end of the period, the gross hours from the beginning of the period or of commercial operation, whichever comes last, to the end of the period or decommissioning, whichever comes first.
HOURS REACTOR CRITICAL	The total clock hours in the report period during which the reactor sustained a controlled chain reaction.
MAXIMUM DEPENDABLE CAPACITY (GROSS) (MDC Gross) (Gross MWe)	Dependable main-unit gross capacity, winter or summer, whichever is smaller. The dependable capacity varies because the unit efficiency varies during the year due to cooling water temperature variations. It is the gross electrical output as measured at the output terminals of the turbine generator during the most restrictive seasonal conditions (usually summer).
MAXIMUM DEPENDABLE CAPACITY (NET) (MDC Net) (Net MWe)	Maximum Dependable Capacity (Gross) less the normal station service loads.
NAMEPLATE RATING (Gross MWe)	The nameplate power designation of the generator in megavolt amperes (MVA) times the nameplate power factor of the generator. NOTE: The nameplate rating of the generator may not be indicative of the maximum or dependable capacity, since some other item of equipment of a lesser rating (e.g., turbine) may limit unit output.
NET ELECTRICAL ENERGY GENERATED	Gross electrical output of the unit measured at the output terminals of the turbine generator during the reporting period, minus the normal station service electrical energy utilization. If this quantity is less than zero, a negative number should be recorded.
OUTAGE	A situation in which no electrical production takes place.
OUTAGE DATE	As reported on Appendix D of Reg. Guide 1.16, the date of the start of the outage. If continued from a previous month, report the same outage date but change "Method of Shutting Down Reactor" to "4 (Other)" and add a note: "Continued from previous month."
OUTAGE DURATION	The total clock hours of the outage measured from the beginning of the report period or the outage, whichever comes last, to the end of the report period or the outage, whichever comes first.
OUTAGE NUMBER	A number unique to the outage assigned by the licensee. The same number is reported each month in which the outage is in progress. One format is "76-05" for the fifth outage to occur in 1976.
PERIOD HOURS	See "Hours in Reporting Period."
POWER REDUCTION	A reduction in the Average Daily Power Level of more than 20% from the previous day. All power reductions are defined as outage of zero hours durations for the purpose of computing unit service and availability factors, and forced outage rate.
REACTOR AVAILAPLE HOURS	The total clock hours in the report period during which the reactor was critical or was capable of being made critical. (Reactor Reserve Shutdown Hours + Hours Reactor Critical.)
REACTOR AVAILABILITY FACTOR	$\frac{\text{Reactor Available Hours} \times 100}{\text{Period Hours}}$
REACTOR RESERVE SHUTDOWN	The cessation of criticality in the reactor for administrative or other similar reasons when operation could have been continued.
REACTOR RESERVE SHUTDOWN HOURS	The total clock hours in the report period during which the reactor is in reserve shutdown mode. NOTE: No credit is given for NRC imposed shutdowns.
REACTOR SERVICE FACTOR	$\frac{\text{Hours Reactor Critical} \times 100}{\text{Period Hours}}$
REPORT PERIOD	Usually, the preceding calender month. Can also be the preceding calendar year, (Year-to-Date), or the life-span of a unit (cumulative).
RESTRICTED POWER LEVEL	Maximum net electrical generation to which the unit is restricted during the report period due to the state of equipment, external conditions, administrative reasons, or a direction by NRC.
SCHEDULED OUTAGE	Planned removal of a unit from service for refueling, inspection, training, or maintenance. Those outages which do not fit the definition of "Forced Outage" perforce are "Scheduled Outages."
STARTUP AND POWER ASCENSION TEST PHASE	Period following initial criticality during which the unit is tested at successively higher levels, culminating with operation at full power for a sustained period and completion of warranty runs. Following this phase, the utility generally considers the unit to be available for commercial operation.
UNIT	The set of equipment uniquely associated with the reactor, including turbine generators, and ancillary equipment, considered as a single electrical energy production facility.
UNIT AVAILABLE HOURS	The total clock hours in the report period during which the unit operated on-line or was capable of such operation. (Unit Reserve Shutdown Hours + Hours Generator On-Line.)
UNIT AVAILABILITY FACTOR*	$\frac{\text{Unit Available Hours} \times 100}{\text{Period Hours}}$
UNIT CAPACITY FACTORS	
– Using Licensed Thermal Power	$\frac{\text{Gross Thermal Energy Generated} \times 100}{\text{Period Hours} \times \text{Lic. Thermal Power}}$
– Using Nameplate Rating	$\frac{\text{Gross Electrical Energy Generated} \times 100}{\text{Period Hours} \times \text{Nameplate Rating}}$
– Using DER	$\frac{\text{Net Electrical Energy Generated} \times 100}{\text{Period Hours} \times \text{DER}}$
– Using MDC Gross	$\frac{\text{Gross Electrical Energy Generated} \times 100}{\text{Period Hours} \times \text{MDC Gross}}$
– Using MDC Net	$\frac{\text{Net Electrical Energy Generated} \times 100}{\text{Period Hours} \times \text{MDC Net}}$

NOTE: if MDC Gross and/or MDC Net have not been determined, the DER is substituted for this quantity for Unit Capacity Factor calculations.

UNIT FORCED OUTAGE RATE	$\frac{\text{Forced Outage Hours}}{\text{Unit Service Hours} + \text{Forced Outage Hours}}$
UNIT RESERVE SHUTDOWN	The removal of the unit from on-line operation for economic or other similar reasons when operation could have been continued.
UNIT RESERVE SHUTDOWN HOURS	The total clock hours in the report period during which the unit was in reserve shutdown mode.
UNIT SERVICE FACTOR	$\frac{\text{Unit Service Hours} \times 100}{\text{Period Hours}}$
UNIT SERVICE HOURS	See "Hours Generator On-Line."

INDEX TO LICENSED UNITS

ARKANSAS 1	A-3
ARKANSAS 2	A-5
BEAVER VALLEY 1	B-3
BIG ROCK POINT 1	B-5
BROWNS FERRY 1	B-7
BROWNS FERRY 2	B-9
BROWNS FERRY 3	B-11
BRUNSWICK 1	B-13
BRUNSWICK 2	B-15
CALVERT CLIFFS 1	C-3
CALVERT CLIFFS 2	C-5
COOK 1	C-7
COOK 2	C-9
COOPER STATION	C-11
CRYSTAL RIVER 3	C-13
DAVIS-BESSE 1	D-3
DRESDEN 1	D-5
DRESDEN 2	D-7
DRESDEN 3	D-9
DUANE ARNOLD	D-11
FARLEY 1	F-3
FITZPATRICK	F-5
FORT CALHOUN 1	F-7
FORT ST VRAIN	F-9
GINNA	G-3
HADDAM NECK	H-3
HATCH 1	H-5
HATCH 2	H-7
HUMBOLDT BAY	H-9
INDIAN POINT 1	I-3
INDIAN POINT 2	I-5
INDIAN POINT 3	I-7
KEWAUNEE	K-3
LA CROSSE	L-3
MAINE YANKEE	M-3
MILLSTONE 1	M-5
MILLSTONE 2	M-7
MONTICELLO	M-9
NINE MILE POINT 1	N-3
NORTH ANNA 1	N-5
OCONEE 1	O-3
OCONEE 2	O-5
OCONEE 3	O-7
OYSTER CREEK 1	O-9
PALISADES	P-3
PEACH BOTTOM 2	P-5
PEACH BOTTOM 3	P-7
PILGRIM 1	P-9
POINT BEACH 1	P-11
POINT BEACH 2	P-13
PRAIRIE ISLAND 1	P-15
PRAIRIE ISLAND 2	P-17
QUAD CITIES 1	Q-3
QUAD CITIES 2	Q-5
RANCHO SECO 1	R-3
ROBINSON 2	R-5
SALEM 1	S-3
SAN ONOFRE 1	S-5
ST LUCIE 1	S-7
SURRY 1	S-9
SURRY 2	S-11
THREE MILE ISLAND 1	T-3
THREE MILE ISLAND 2	T-5
TROJAN	T-7
TURKEY POINT 3	T-9
TURKEY POINT 4	T-11
VERMONT YANKEE 1	V-3
YANKEE-ROWE 1	Y-3
ZION 1	Z-3
ZION 2	Z-5

SECTION 1 – CURRENT DATA SUMMARIES

MONTHLY HIGHLIGHTS OF COMMERCIAL NUCLEAR POWER UNITS

FOR MONTH OF

DECEMBER 1979

LICENSED POWER REACTORS:

NUMBER		CAPACITY[1] MWe (Net)
66	IN COMMERCIAL OPERATION	48,151
1	NOT IN COMMERCIAL OPERATION	912
67[2]	LICENSED TO OPERATE	49,063

POWER GENERATION

	REPORT MONTH	PREVIOUS MONTH	YEAR-TO-DATE
1. GROSS ELECTRICAL (MWHE)	21,468,763	20,011,593	264,375,005
2. NET ELECTRICAL (MWHE)	20,474,669	19,038,225	251,941,721
3. AVG. UNIT SERVICE FACTOR (%)	66.4	65.7	66.5
4. AVERAGE UNIT AVAILABILITY FACTOR (%)	66.6	66.1	67.9
5. AVERAGE UNIT CAPACITY FACTOR (MDC) (%)	60.7	58.3	60.5
6. AVG. UNIT CAPACITY FACTOR (DER) (%)	59.2	56.7	58.9
7. FORCED OUTAGE RATE (%)	13.0	15.1	14.3

NOTES

[1] BASED UPON MAXIMUM DEPENDABLE CAPACITY; DESIGN ELECTRIC RATING USED IF MDC NOT DETERMINED.

[2] EXCLUDES INDIAN POINT 1 (DER = 265), HUMBOLDT BAY (DER = 65), AND THREE MILE ISLAND 2 (DER = 906) WHICH HAVE OPERATING LICENSES BUT ARE SHUTDOWN INDEFINITELY.

ACTUAL VS. POTENTIAL ENERGY PRODUCTION

			% OF POTENTIAL PRODUCTION
POTENTIAL ENERGY PRODUCTION IN THIS PERIOD BY UNITS IN COMMERCIAL OPERATION (Using Maximum Dependable Capacity Net)		35,824,344 MWHe	
ENERGY ACTUALLY PRODUCED DURING THIS REPORT PERIOD	=	21,468,763 MWHe (GROSS) (20,474,669) (NET)	59.9 (57.2)
ENERGY NOT PRODUCED DURING THIS REPORT PERIOD DUE TO SCHEDULED OUTAGES (NET)	=	5,721,935 MWHe (7,791,268)	16.0 (21.7)
ENERGY NOT PRODUCED DURING THIS REPORT PERIOD DUE TO FORCED OUTAGES (NET)	=	3,758,547 MWHe (5,483,469)	10.5 (15.3)
ENERGY NOT PRODUCED DURING THIS REPORT PERIOD FOR OTHER REASONS (NET)	=	4,875,099 MWHe (2,074,938)	13.6 (5.8)
TOTAL		35,824,344 MWHe =	100%
ENERGY NOT PRODUCED THIS REPORT PERIOD DUE TO NRC RESTRICTIONS	=	3,970,964 MWHe	11.1
NUMBER OF UNITS WITH POWER RESTRICTIONS IMPOSED BY NRC	=	1	

NOTE: The numbers shown in parentheses ( ) represent a different method of computing nuclear energy distribution for the month. Both sets of statistics will be published for the remainder of the year so a comparison between methods of calculations can be made. Starting with the month of January 1980, the new method of calculation will be used exclusively.

The changes in method of calculation are:

Energy actually produced during this report period - this parameter used to list the gross electrical production for the month. The new calculation uses the net electrical energy.

Energy not produced during this report period due to scheduled and forced outages - the existing method for allocating numerical values to these parameters was the summation of the product of the unit MDC net times the length of the outage times the capacity factor of the unit for the month. A capacity factor of 0.75 was substituted for units that did not operate during the month. The new method of calculation will be the summation of the products of the unit MDC net times the length of the outage.

The remaining computations will remain the same, however, the amount of energy allocation will change because of differences in the previous computations. For instance; energy not produced during this report for other reasons will still be the residual after the other three energy parameters (energy produced, forced & scheduled outages) are subtracted from the potential power production.

Energy not produced this report period due to NRC restrictions has been and will continue to be the summation of the products of unit MDC times the length of the regulatory outage plus the summation of the reduction in power caused by the regulatory restriction times the length of the NRC restriction, if applicable.

OUTAGE DATA

	NUMBER	HOURS	PERCENT OF CLOCK TIME	MWHE LOST PRODUCTION
1. FORCED OUTAGES DURING MONTH	42	6,381.6	13.0	3,758,547
2. SCHEDULED OUTAGES DURING THE MONTH	32	10,097.4	20.6	5,721,935
3. TOTAL	74	16,479.0	33.6	9,480,482

Down time x licensed power (MWe) x capacity factor for report month

Capacity factor of 0.75 was assumed for units that did not operate at all during the period.

NOTE: ENTRIES IN THE "MWHE LOST PRODUCTION" COLUMN WILL CONTINUE TO BE PUBLISHED USING THE TRADITIONAL METHOD OF CALCULATION. STARTING WITH JANUARY 1980 DATA, THE REVISED METHOD OF CALCULATING THESE PARAMETERS, AS SHOWN IN PARENTHESES AND DESCRIBED IN THE "ACTUAL VS. POTENTIAL ENERGY PRODUCTION" SECTION WILL BE PUBLISHED EXCLUSIVELY.

DERATED UNITS

UNIT NAME	MAXIMUM DEPENDABLE CAPACITY (MWe - NET)	POWER LIMIT (MWe - NET)
DRESDEN 3	773	550
†FORT ST VRAIN	330	231
POINT BEACH 1	495	390
ROBINSON 2	665	635

†Restricted by NRC

MONTHLY HIGHLIGHTS OF COMMERCIAL NUCLEAR POWER UNITS

FOR MONTH OF

DECEMBER 1979

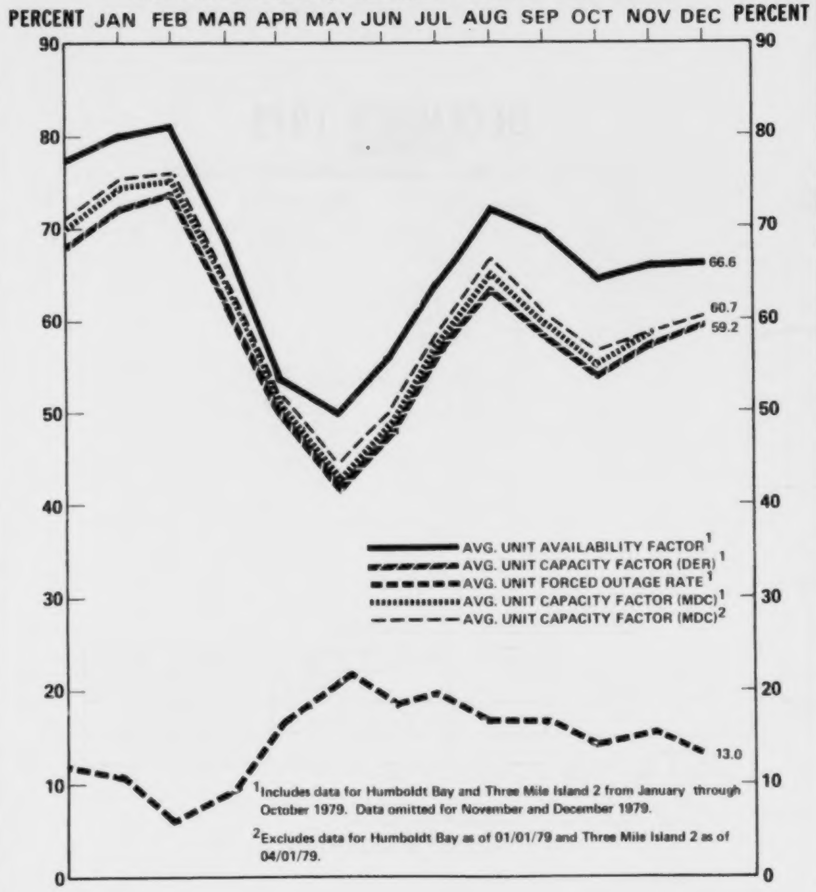
REASONS FOR SHUTDOWN

REASON	NUMBER	TOTAL HOURS LOST
A - Equipment Failure	20	2,144.8
B - Maintenance or Test	15	2,025.9
C - Refueling	12	7,631.0
D - NRC Restriction	14	4,053.7
E - Operator Training and License Examination	0	0
F - Administrative	0	0
G - Operational Error '	5	28.4
H - Other	8	595.2
TOTAL	74	16,479.0

SHUTDOWNS GREATER THAN 72 HOURS EACH

UNIT NAME	REASON	UNIT NAME	REASON
BEAVER VALLEY 1	C	MILLSTONE 2	H
BROWNS FERRY 2	H	NORTH ANNA 1	C
BROWNS FERRY 3	C	OCONEE 1	C
BRUNSWICK 1	A,D	PALISADES	C
BRUNSWICK 2	D	PEACH BOTTOM 3	B
CALVERT CLIFFS 2	B	POINT BEACH 1	A
COOK 1	H	QUAD CITIES 1	B
COOK 2	C	QUAD CITIES 2	C
DAVIS–BESSE 1	B	SALEM 1	2D
DRESDEN 1	C	SURRY 1	A
FITZPATRICK	H	SURRY 2	C
FORT CALHOUN 1	A	THREE MILE ISLAND 1	D
FORT ST VRAIN	B	TROJAN	D
GINNA	A	TURKEY POINT 3	C
HATCH 1	A	TURKEY POINT 4	A
INDIAN POINT 2	A	ZION 1	D
INDIAN POINT 3	C	ZION 2	D

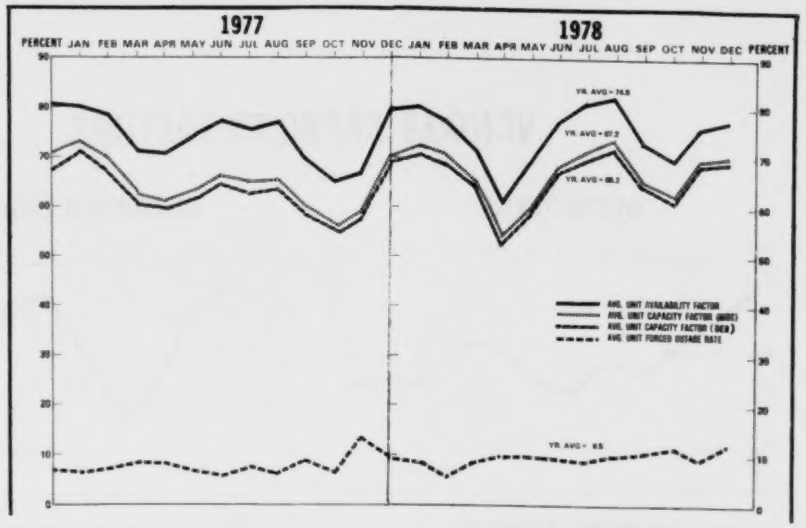
UNIT AVAILABILITY, CAPACITY AND FORCED OUTAGE



[1] Includes data for Humboldt Bay and Three Mile Island 2 from January through October 1979. Data omitted for November and December 1979.

[2] Excludes data for Humboldt Bay as of 01/01/79 and Three Mile Island 2 as of 04/01/79.

UNIT AVAILABILITY, CAPACITY AND FORCED OUTAGE

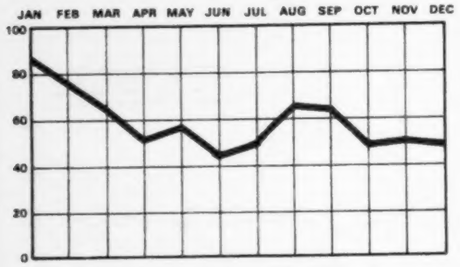


REFUELING SCHEDULE

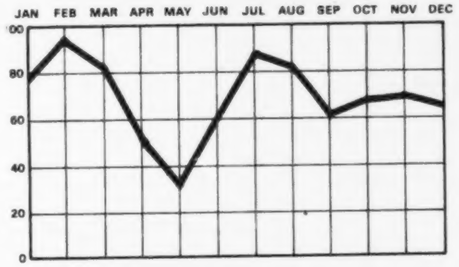
Please refer to the table on page 3–6 for the scheduled refueling date for each nuclear powerplant in this publication.

VENDOR CAPACITY FACTORS

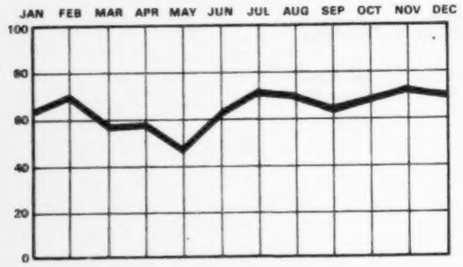
WESTINGHOUSE



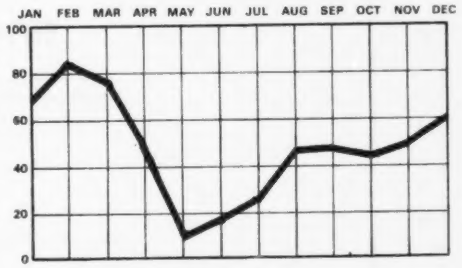
COMBUSTION ENGINEERING



GENERAL ELECTRIC



BABCOCK & WILCOX



NOTE: GRAPH INCLUDES THREE MILE ISLAND 2 THROUGH OCTOBER 1979.

DECEMBER 1979

OPERATIONS

1. COMPUTED BY ARITHMETIC AVERAGE OF UNIT CAPACITY FACTORS

GE BWR's	CFMDC
BROWNS FERRY 1	86.2
BROWNS FERRY 2	58.3
BROWNS FERRY 3	49.3
BRUNSWICK 1	51.3
BRUNSWICK 2	74.4
COOPER STATION	93.0
DRESDEN 1	.0
DRESDEN 2	87.7
DRESDEN 3	57.6
DUANE ARNOLD	90.1
FITZPATRICK	80.1
HATCH 1	29.9
HATCH 2	105.6
MILLSTONE 1	89.5
MONTICELLO	89.8
NINE MILE POINT 1	98.4
OYSTER CREEK 1	91.9
PEACH BOTTOM 2	95.8
PEACH BOTTOM 3	74.8
PILGRIM 1	93.2
QUAD CITIES 1	70.5
QUAD CITIES 2	.0
VERMONT YANKEE 1	102.6
23 UNITS	72.6

W PWR's	CFMDC
BEAVER VALLEY 1	.0
COOK 1	71.2
COOK 2	.0
FARLEY 1	87.9
GINNA	43.5
HADDAM NECK	105.3
INDIAN POINT 2	63.9
INDIAN POINT 3	.0
KEWAUNEE	97.7
NORTH ANNA 1	.0
POINT BEACH 1	42.5
POINT BEACH 2	95.2
PRAIRIE ISLAND 1	92.5
PRAIRIE ISLAND 2	96.2
ROBINSON 2	99.2
SALEM 1	.0
SAN ONOFRE 1	97.8
SURRY 1	58.2
SURRY 2	.0
TROJAN	.0
TURKEY POINT 3	.0
TURKEY POINT 4	73.4
YANKEE-ROWE 1	99.8
ZION 1	.0
ZION 2	.0
25 UNITS	49.0

CE PWR's	CFMDC
CALVERT CLIFFS 1	52.0
CALVERT CLIFFS 2	73.1
FORT CALHOUN 1	79.3
MAINE YANKEE	78.9
MILLSTONE 2	80.0
PALISADES	0.0
ST LUCIE 1	100.9
7 UNITS	66.3

B&W PWR's	CFMDC
ARKANSAS 1	98.2
CRYSTAL RIVER 3	94.5
DAVIS-BESSE 1	.0
OCONEE 1	.0
OCONEE 2	95.7
OCONEE 3	98.4
RANCHO SECO 1	98.0
THREE MILE ISLAND 1	.0
8 UNITS	60.6

All PWR's	CFMDC
40 UNITS	54.3

2. COMPUTED BY RATIO OF VENDOR TOTALS

	GE BWR's	W PWR's	CE PWR's	B&W PWR's	All PWR's
NET ELECTRICAL PRODUCTION	9,447,001	5,307,632	2,604,772	3,042,835	10,955,239
MDC NET	17,327	18,435	5,178	6,768	30,381
CFMDC	73.3	38.7	67.6	60.4	48.5

Capacity factor in this page, denoted as CFMDC, is a function of the net maximum dependable capacity. See the corresponding definition in the Glossary. The vendor averages are either computed as (1) the arithmetic average of the stated CFMDC for that vendor or (2) by the formula:

$$\frac{\text{Net Electrical Energy Produced by Vendor}}{\text{Potential Electrical Production By Vendor In This Month.}} \times 100\%$$

Units Excluded	Reason for Exclusion
Indian Point 1	Unit Shutdown Indefinitely
Humboldt Bay	
Three Mile Island 2	
Arkansas 2	Not in Commercial Operation
Big Rock Point	Not Representative of Typical BWRs
LaCrosse	Not Representative of Typical BWRs and Atypical Vendor
Fort St. Vrain	Atypical Vendor and Reactor Design

AVERAGE DAILY POWER LEVEL FOR ALL COMMERCIALLY OPERATING UNITS

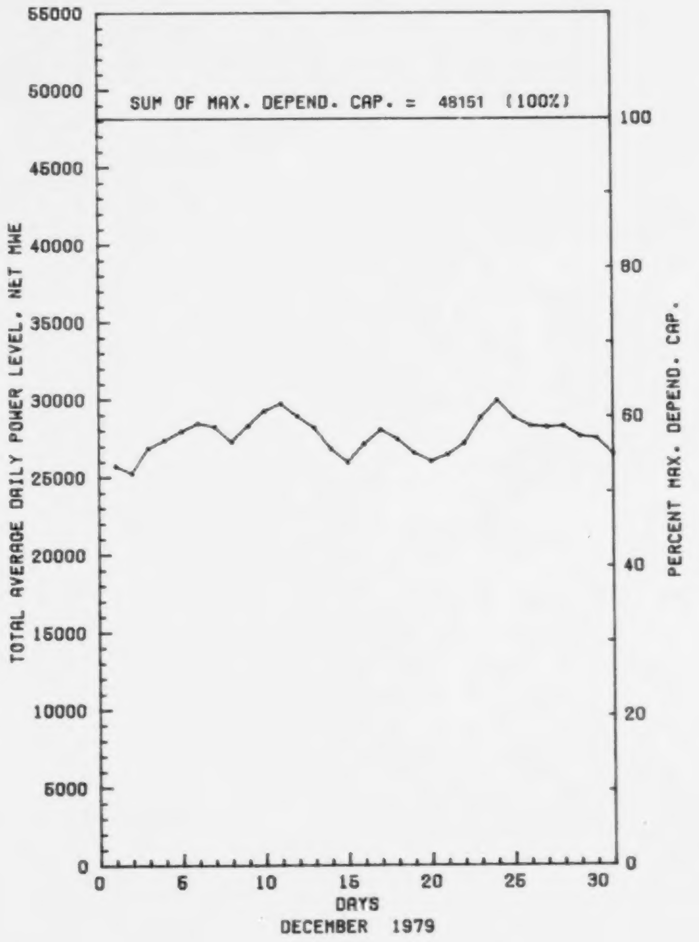
This chart depicts the average daily power level for the units in commercial operation during the month.

The straight line on the graph labelled "SUM OF MAX DEP CAP'S" is plotted at the value shown by summing the separate maximum dependable capacities of the commercially operating units (in Net MWe). The plot shown below the line is calculated by summing the separate average daily power levels of the commercially operating units for each day of the month.

The scale on the left vertical axis runs in 1,000 MWe increments from 0 to 55,000 MWe (Net). The right vertical axis shows the percentage in 10% increments, up to 100% of the "SUM OF MAX DEP CAP'S."

It should be recognized that the 100% line would be obtainable only if all of the commercially operating units operated at 100% capacity, 24 hours per day, for the entire month. In other words, since any power generator must occasionally shut down to refuel and/or perform needed maintenance, and also since 100% capacity production is not always required by power demands, the 100% line is a theoretical goal and not a practical one.

SUMMARY PLOT FOR COMMERCIALLY OPERATING UNITS



ERRATA

Corrections and Amendments to Previously-Reported Data

(NOTE: These changes are reflected in the data contained in the current report)

CORRECTIONS TO NUREG 0020, VOL. 3 NO. 12 DECEMBER 1979

Page	Unit Name	Item	Correction
	NONE		

MULTIPLE - ISSUE CORRECTIONS

MAINE YANKEE	Item 16. Gross Thermal Energy (MWH) in the cumulative column has been decreased by 25 MWH because of a long-term licensee audit.
RANCHO SECO	Item 16. Gross Thermal Energy (MWH); a revision in the burnup coefficient has resulted in a decrease of 524,381 in the year-to-date column and 528.113 in the cumulative column.

SECTION 2 – OPERATING POWER REACTORS

ARKANSAS 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Arkansas/Pope 6 miles NW of Russelville
2.	Electric Reliability Council	Member of SW Power Pool
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2568
5.	Design Electrical Rating (MWE Net)	850
6.	Date of Initial Criticality	August 6, 1974
7.	Date of Commercial Operation	December 19, 1974
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Dardenelle Reservoir

Utility & Contractor Information

10.	Licensee	Arkansas Power and Light Company
11.	Corporate Address	P.O. Box 551 Little Rock, Arkansas 72203
12.	Corporate Contact	W. Cavanaugh III Vice Pres., Generation & Construction
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	B&W
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	IV
17.	IE Branch Chief	G. L. Madsen
18.	IE Senior Inspector	T. F. Westerman
19.	IE Principal Inspector	W. D. Johnson
20.	Licensing Project Manager	G. S. Vissing
21.	Docket Number	50-313
22.	License Number/Date of Issue	DPR-51, 5/21/74
23.	Public Document Room	Arkansas Polytechnic College Russelville, Arkansas 72801

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO 10 CFR 50, APPENDIX B, CRITERIA V, SECTION 5 OF THE QUALITY ASSURANCE TOPICAL REPORT (APL-TOP-1A, REV. 4), AND A LICENSEE PROCEDURE 1004.23 ON DRAWING DOCUMENT CONTROL, A SAFETY RELATED AS-BUILT DRAWING M-231, WAS NOT REVISED AND ISSUED FOLLOWING IMPLEMENTATION OF DESIGN CHANGE REQUEST 79-1001, SMALL BREAK ECCS REDESIGN.
(7916 INFRACTION)

CONTRARY TO TS 3.1.1.3.A, THE REACTOR WAS OPERATED AFTER THE PRESSURIZER CODE SAFETY VALVE PSV-1002 WAS TESTED ON MARCH 24, 1978 AND LEFT WITH A SET POINT IN EXCESS OF THE ALLOWABLE SET POINT IN THE ACCEPTANCE CRITERIA OF THE TEST PROCEDURE AND AS STATED IN THE BASES OF TS 3.1.
(7916 INFRACTION)

CONTRARY TO TS 6.4.1 AND SECTION 5.5 OF ANSI N18.1, (1) THE LICENSEE HAD NOT FULLY IMPLEMENTED THE AND TRAINING PLAN (PROGRAM), (2) THE TRAINING PLAN CONTAINED NO RETRAINING REQUIREMENTS FOR SEVERAL GROUPS OF EMPLOYEES, AND (3) THE TRAINING PLAN PROVIDED NO MEANS FOR EVALUATING THE EFFECTIVENESS OF THE TRAINING PLAN.
(7916 INFRACTION)

UNIT 1 TS 6.5.1.7.1.B AND UNIT 2 TS 6.5.1.7.B REQUIRES THE PLANT SAFETY COMMITTEE (PSC) TO RENDER DETERMINATIONS IN WRITING WITH REGARD TO WHETHER OR NOT PROCEDURES REVIEWED IN ACCORDANCE WITH UNIT 1 AND UNIT 2 TS 6.5.1.6.A CONSTITUTE AN UNREVIEWED SAFETY QUESTION. CONTRARY TO THE ABOVE, THE PSC DID NOT MAKE WRITTEN SAFETY EVALUATIONS OR RENDER DETERMINATIONS IN WRITING WITH REGARD TO WHETHER OR NOT THE PROCEDURES REVIEWED CONSTITUTED AN UNREVIEWED SAFETY QUESTION.
(7916 INFRACTION)

UNIT 1 AND UNIT 2 TS 6.5.2.7A REQUIRES THE SAFETY REVIEW COMMITTEE (SRC) TO REVIEW THE SAFETY EVALUATIONS FOR CHANGES TO EQUIPMENT OR SYSTEMS COMPLETED UNDER THE PROVISIONS OF 10 CFR 50.59 AND TO VERIFY THAT SUCH ACTIONS DID NOT CONSTITUTE AN UNREVIEWED SAFETY QUESTION. ADDITIONALLY, UNIT 1 AND UNIT 2 TS 6.5.2.1 REQUIRES THE SRC TO PERFORM THE ABOVE DESCRIBED REVIEW INDEPENDENTLY. CONTRARY TO THE ABOVE, INDEPENDENT REVIEWS HAD NOT BEEN PERFORMED. THE MANAGER OF LICENSING HAD IN-LINE RESPONSIBILITY FOR PREPARING THE INITIAL 10 CFR 50.59 REVIEW ON DESIGN CHANGES AND WAS ASSIGNED TO PERFORM THE SRC INDEPENDENT REVIEW OF THE SAME EVALUATIONS.
(7916 INFRACTION)

UNIT 2 TS 6.5.2.8.B REQUIRES SRC AUDITS TO ENCOMPASS THE PERFORMANCE, TRAINING, AND QUALIFICATIONS OF THE ENTIRE UNIT STAFF ONCE A YEAR. CONTRARY TO THE ABOVE, THE SRC DID NOT AUDIT NON-LICENSED TRAINING.
(7916 INFRACTION)

CONTRARY TO 10 CFR 19.12, THE LICENSEE HAD NOT INSTRUCTED ITS EMPLOYEES IN THEIR RESPONSIBILITY TO PROMPTLY REPORT TO THE LICENSEE ANY CONDITION WHICH MAY RESULT IN A VIOLATION OF REGULATORY REQUIREMENTS OR UNNECESSARY EXPOSURE TO RADIATION OR RADIOACTIVE MATERIAL.
(7916 DEFICIENCY)

CONTRARY TO ADMINISTRATIVE PROCEDURE 1005.17, CONTROL OF COMBUSTIBLES, THE INSPECTOR FOUND THAT CORRIDOR 98, ELEVATION 372' IN THE AUXILIARY BUILDING CONTAINED A BARREL HALF FULL OF MISCELLANEOUS COMBUSTIBLE TRASH AND A ONE GALLON SAFETY CAN CONTAINING SOLVENT.
(7922 INFRACTION)

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
79-21 03L-0	11/15/79	12/6/79	Containment isolation valve failure
79-22 03L-0	11/19/79	12/18/79	Failure to perform required surveillance on steam driven emergency feedwater pump

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

FACILITY ITEMS (PLANS AND PROCEDURES):

MANAGERIAL ITEMS

PLANT STATUS:

The plant is at 100 percent power with plans to shut down on 31 December in order to perform design changes required by NUREG 0578.

LAST IE SITE INSPECTION DATE: October 6 - November 21, 1979

INSPECTION REPORT NO: 50-313/79-22

SUMMARY

INSPECTION SUMMARY

Inspection conducted during period of September 9 through October 5, 1979 (79-21): Routine, announced inspection of the QA/QC Administrative Program, licensee actions taken in response to IE Bulletins, and follow-up on open items. One item of noncompliance was identified (deficiency - outdated Q-lists).

Inspection on November 5 - 9, (79-23): Routine, unannounced inspection of the licensee's program for control of maintenance and preventive maintenance. No items of noncompliance or deviations were identified.

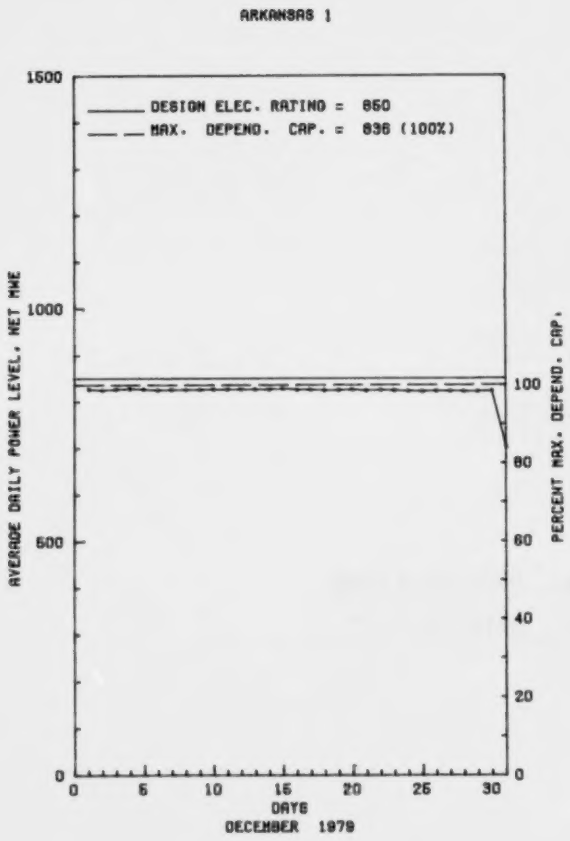
Inspection conducted during period of October 6 through November 21, 1979 (79-22): Routine, announced inspection of previously identified inspection items, plant operations, surveillance testing and calibration control program, and personnel qualification program. One item of noncompliance was identified (infraction - control of combustibles).

REVIEWED BY OMPA LOEB: DATE: 2/11/80

REVIEWED BY NRR DOR: DATE: 2/13/80

ARKANSAS 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-313
2. Utility Contact: L. S. Bramlett (501) 968-2519
3. Licensed Thermal Power (MWt): 2568
4. Nameplate Rating (Gross MWe): 1003 x 0.9 = 903
5. Design Electrical Rating (Net MWe): 850
6. Maximum Dependable Capacity (Gross MWe): 883
7. Maximum Dependable Capacity (Net MWe): 836
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	8,760.0	44,131.0
12. Hours Reactor Was Critical	742.8	4,428.1	30,138.2
13. Reactor Reserve Shutdown Hours	.0	2,257.5	4,247.4
14. Hours Generator On-Line	742.2	4,256.8	29,470.5
15. Unit Reserve Shutdown Hours	.0	591.5	796.7
16. Gross Thermal Energy (MWH)	1,897,338	10,455,335	71,262,844
17. Gross Electrical Energy (MWH)	638,300	3,480,475	23,721,156
18. Net Electrical Energy (MWH)	611,079	3,323,490	22,634,036
19. Unit Service Factor	99.8	48.6	66.8
20. Unit Availability Factor	99.8	55.3	68.6
21. Unit Capacity Factor (MDC Net)	98.2	45.4	61.3
22. Unit Capacity Factor (DER Net)	96.6	44.6	60.3
23. Unit Forced Outage Rate	.0	35.7	15.6

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 02/18/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
79-06	12/31/79	S	1.8	D	1		CH	INSTRU	COMMITMENTS TO NRC TO PROVIDE MODIFICATIONS DUE TO TMI 2.

SUMMARY

OPERATION WAS AT NEAR FULL POWER DURING THE REPORT PERIOD, WITH AN OUTAGE ON 12/31/79 DUE TO COMMITMENTS TO NRC TO PROVIDE MODIFICATIONS DUE TO TMI-2.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuation
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

ARKANSAS 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Arkansas/Pope 6 miles NW of Russelville
2.	Electric Reliability Council	Member of SW Power Pool
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2815
5.	Design Electrical Rating (MWE Net)	912
6.	Date of Initial Criticality	12-5-78
7.	Date of Commercial Operation	N/A
8.	Condenser Cooling Method	Recirculating
9.	Condenser Cooling Water Source	Cooling Tower

Utility & Contractor Information

10.	Licensee	Arkansas Power & Light Company
11.	Corporate Address	P.O. Box 551 Little Rock, Arkansas 72203
12.	Corporate Contact	W. Cavanaugh III Vice Pres., Generation & Construction
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Combustion Engineering, Inc.
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	IV
17.	IE Branch Chief	G. L. Madsen
18.	IE Senior Inspector	T. F. Westerman
19.	IE Principal Inspector	W. D. Johnson
20.	Licensing Project Manager	G. S. Vissing
21.	Docket Number	50-368
22.	License Number/Date of Issue	NFP-6, 9/1/78
23.	Public Document Room	Arkansas Polytechnic College Russelville, Arkansas 72801

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO TS 6.4.1 AND SECTION 5.5 OF ANSI N18.1-1971, (1) THE LICENSEE HAD NOT FULLY IMPLEMENTED THE ANO TRAINING PLAN (PROGRAM), (2) THE TRAINING PLAN CONTAINED NO RETRAINING REQUIREMENTS FOR SEVERAL GROUPS OF EMPLOYEES, AND (3) THE TRAINING PLAN PROVIDED NO MEANS FOR EVALUATING THE EFFECTIVENESS OF THE TRAINING PLAN.
(7914 INFRACTION)

UNIT 1 TS 6.5.1.7.1.B AND UNIT 2 TS 6.5.1.7.B REQUIRES THE PLANT SAFETY COMMITTEE (PSC) TO RENDER DETERMINATIONS IN WRITING WITH REGARD TO WHETHER OR NOT PROCEDURES REVIEWED IN ACCORDANCE WITH UNIT 1 AND UNIT 2 TS 6.5.1.6.A CONSTITUTE AN UNREVIEWED SAFETY QUESTION. CONTRARY TO THE ABOVE, THE PSC DID NOT MAKE WRITTEN SAFETY EVALUATIONS OR RENDER DETERMINATIONS IN WRITING WITH REGARD TO WHETHER OR NOT THE PROCEDURES REVIEWED CONSTITUTED AN UNREVIEWED SAFETY QUESTION.
(7914 INFRACTION)

UNIT 1 AND UNIT 2 TS 6.5.2.7A REQUIRES THE SAFETY REVIEW COMMITTEE (SRC) TO REVIEW THE SAFETY EVALUATIONS FOR CHANGES TO EQUIPMENT OR SYSTEMS COMPLETED UNDER THE PROVISIONS OF 10 CFR 50.59 AND TO VERIFY THAT SUCH ACTIONS DID NOT CONSITITUTE AN UNREVIEWED SAFETY QUESTION. ADDITONALLY, UNIT 1 AND UNIT 2 TS 6.5.2.1 REQUIRES THE SRC TO PERFORM THE ABOVE DESCRIBED REVIEW INDEPENDENTLY. CONTRARY TO THE ABOVE, INDEPENDENT REVIEWS HAD NOT BEEN PERFORMED. THE MANAGER OF LICENSING HAD IN-LINE RESPONSIBILITY FOR PREPARING THE INITIAL 10 CFR 50.59 REVIEW ON DESIGN CHANGES AND WAS ASSIGNED TO PERFORM THE SRC INDEPENDENT REVIEW OF THE SAME EVALUATIONS.
(7914 INFRACTION)

UNIT 2 TS 6.5.2.8.B REQUIRES SRC AUDITS TO ENCOMPASS THE PERFORMANCE, TRAINING, AND QUALIFICATIONS OF THE ENTIRE UNIT STAFF ONCE A YEAR. CONTRARY TO THE ABOVE, THE SRC DID NOT AUDIT NON-LICENSED TRAINING.
(7914 INFRACTION)

CONTRARY TO 10 CFR 19.12, THE LICENSEE HAD NOT INSTRUCTED ITS EMPLOYEES IN THEIR RESPONSIBILITY TO PROMPTLY REPORT TO THE LICENSEE ANY CONDITION WHICH MAY RESULT IN A VIOLATION OF REGULATORY REQUIREMENTS OR UNNECESSARY EXPOSURE TO RADIATION OR RADIOACTIVE MATERIAL.
(7914 DEFICIENCY)

CONTRARY TO ADMINISTRATIVE PROCEDURE QCP 1004.02, INITIATING AND PROCESSING OF TROUBLE REPORTS, THE DELTA-T SIGMA WEATHER STATION RECORDER (TR-9300) WAS INOPERABLE FOR A PERIOD OF ABOUT 35 HOURS BETWEEN OCTOBER 16 AND OCTOBER 18, 1979, AND OPERATIONS PERSONNEL DID NOT OBSERVE AND REPORT THE MALFUNCTION, EVEN THOUGH THE TECHNICAL SPECIFICATION DAILY CHANNEL CHECK ON THIS INSTRUMENT WAS PERFORMED TWICE DURING THIS PERIOD.
(7921 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

+ The 4 RTD's on the west side of the hot legs normally indicate a temperature about 3 degrees F higher than the 4 RTD's on the east side of the hot legs at 50 % power. This relationship occasionally reverses. A design change to minimize the effect of the anomaly on the Core Protection Calculators has been installed.

FACILITY ITEMS (PLANS AND PROCEDURES):

MANAGERIAL ITEMS:

PLANT STATUS:

+ The unit conducted post design change and power ascension testing during most of the month.

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
79-87 03L-0	11/14/79	12/6/79	Low pressure safety injection flow orifice valve socket weld leak

LAST IE SITE INSPECTION DATE: October 6 - November 21, 1979 +

INSPECTION REPORT NO: 50-368/79-21 +

SUMMARY

INSPECTION SUMMARY

Inspection conducted during period of September 9 - October 5, 1979 (79-20): Routine, announced inspection of the QA/QC Administrative Program, licensee actions taken in response to IEB 79-04, licensee actions taken in response to IEB 79-14, and testing associated with investigating the hot leg temperature anomaly. Within the four areas inspected, no items of noncompliance were identified.

Inspection conducted during period of November 5 - 9, 1979 (79-22): Routine, unannounced inspection of the licensee's program for control of maintenance and preventive maintenance. Within the area inspected, no items of noncompliance or deviations were identified.

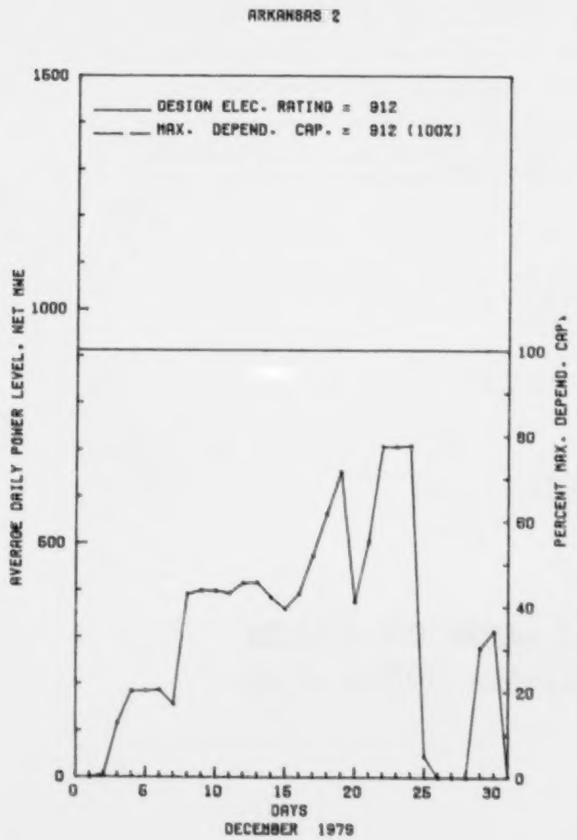
Inspection conducted during period of October 6 - November 21, 1979 (79-21): Routine, announced inspection of previously identified inspection items, plant operations, surveillance testing and calibration control program, and personnel qualification program. Within the four areas inspected, two items of noncompliance were identified (infraction - failure to adhere to procedure, and infraction - physical barrier).

REVIEWED BY OMPA LOEB: MR Beebe DATE: 02/11/80

REVIEWED BY NRR DOR: DATE: 2/13/80

ARKANSAS 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-368
2. Utility Contact: Rex Pendergraft (501) 968-2519
3. Licensed Thermal Power (MWt): 2815
4. Nameplate Rating (Gross MWe): 959
5. Design Electrical Rating (Net MWe): 912
6. Maximum Dependable Capacity (Gross MWe): TO BE DETERMINED
7. Maximum Dependable Capacity (Net MWe): TO BE DETERMINED
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	8,760.0	9,504.0
12. Hours Reactor Was Critical	605.9	3,393.5	3,828.4
13. Reactor Reserve Shutdown Hours	8.0	2,150.9	2,293.1
14. Hours Generator On-Line	537.1	2,984.1	3,036.9
15. Unit Reserve Shutdown Hours	.0	21.7	21.7
16. Gross Thermal Energy (MWH)	813,274	3,452,233	3,496,811
17. Gross Electrical Energy (MWH)	251,195	977,718	983,285
18. Net Electrical Energy (MWH)	232,884	881,825	885,809
19. Unit Service Factor			
20. Unit Availability Factor			
21. Unit Capacity Factor (MDC Net)			
22. Unit Capacity Factor (DER Net)			
23. Unit Forced Outage Rate			

UNIT NOT IN COMMERCIAL OPERATION

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
79-22	10/04/79	S	43.8	H	2		IF	AIRDRY	OUTAGE CONTINUED FROM PREVIOUS MONTH.
79-23	12/06/79	F	10.6	A	2		CB	VALVEX	PRESSURIZER SPRAY VALVE STUCK OPEN. VALVE WAS REPAIRED AND RETURNED TO SERVICE.
79-24	12/20/79	F	12.6	A	3		HC	HTEXCH	HOTWELL LEVEL FLASHED CAUSING THE LOSS OF A FEEDWATER PUMP. DUMPS GOING TO CONDENSER WERE ADJUSTED TO CORRECT SITUATION.
79-25	12/25/79	F	104.4	A	3				CONDENSER TUBE LEAKS FORCED THE OUTAGE. FAILED TUBES WERE PLUGGED.
79-26	12/30/79	S	35.5	B	3				TRIPPED ALL RCP'S PER T.P. 2800.01 APPENDIX AA.

SUMMARY

UNIT STARTUP TESTING CONTINUED AT THE 80% LEVEL. PROBLEMS OCCURRED WITH A PRESSURIZER SPRAY VALVE, IN THE CONDENSER HOTWELL, AND WITH CONDENSER TUBES. A SCHEDULED OUTAGE OCCURRED AT THE END OF THE MONTH WHEN ALL REACTOR COOLANT PUMPS WERE TRIPPED PER THE STARTUP TESTING SEQUENCE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

NRC POWER RESTRICTIONS: NONE

BEAVER VALLEY 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Pennsylvania/Beaver 5 miles E of E. Liverpool, Ohio
2.	Electric Reliability Council	ECAR
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2652
5.	Design Electrical Rating (MWE Net)	852
6.	Date of Initial Criticality	May 10, 1976
7.	Date of Commercial Operation	10/01/76 (50%) 04/30/77 (100%)
8.	Condenser Cooling Method	Cooling tower/Closed loop
9.	Condenser Cooling Water Source	Ohio River

Utility & Contractor Information

10.	Licensee	Duquesne Light Company
11.	Corporate Address	435 Sixth Avenue Pittsburgh, Pennsylvania 15219
12.	Corporate Contact	C. N. Dunn, Vice President
13.	Architect/Engineer	Stone & Webster
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Senior Inspector	R. R. Keimig
19.	IE Principal Inspector	D. Beckman
20.	Licensing Project Manager	D. L. Wigginton
21.	Docket Number	50-334
22.	License Number/Date of Issue	DPR-66, 1/30/76
23.	Public Document Room	B. F. Jones Memorial Library 633 Franklin Avenue Alquippa, PA 15001

INSPECTION STATUS

ENFORCEMENT STATUS

THERE WERE NO WRITTEN SAFETY ANALYSIS/EVALUATION CONTAINED IN OSC MINUTES 51-4 & 55-5 TO PROVIDE THE BASIS FOR THE OSC DETERMINATIONS REGARDING DC 0223, GENERATOR MANUAL FIELD FLASH & THE ASSOCIATED TEST PROCEDURE & DC 233, SAFETY INSPECTION LINE HANGERS.
(7918 INFRACTION)

AS OF 8/24/79, ISI PROCEDURE 5.8, REACTOR COOLANT SYSTEM HYDRAULIC PIPE SUPPORT EXAMINATION, REV 2, HAD NOT BEEN UPDATED TO REFLECT THE INSTALLATION OF A NEW SAFETY RELATED HYDRAULIC SNUBBER (SI-HS-529) NOR HAD THE LICENSEE SUBMITTED THE REQUIRED TS CHANGE TO NRC REQUESTING ADDITION OF THE HYDRAULIC SNUBBER TO TABLE 3.7.4.
(7918 INFRACTION)

NO PROCEDURE EXISTED WHICH REQUIRED THE ONSITE ENGINEERING GROUP TO FORWARD INFORMATION REGARDING A FIELD CHANGE TO THE CHARGING PUMP MODIFICATION TO THE RESPONSIBLE ENGINEERING GROUP FOR REVIEW. AS A RESULT THE FIELD CHANGE WAS ACCOMPLISHED AND THE PLANT RETURNED TO POWER OPERATION WITHOUT THE REQUIRED SEISMIC RE-ANALYSIS BEING ACCOMPLISHED.
(7918 INFRACTION)

AS OF 8/24/79, A NUMBER OF DRAWINGS WHICH ARE MAINTAINED IN THE STATION FILES DO NOT REFLECT THE AS-BUILT CONDITION AS DESCRIBED IN THE RESPECTIVE DESIGN CHANGES.
(7918 INFRACTION)

ON 8/24/79, A NUMBER OF EXAMPLES OF NONCOMPLIANCE WITH THE ANSI N45.2.9 REQUIREMENTS FOR THE STORAGE OF SINGLE COPY RECORDS WERE IDENTIFIED.
(7918 DEFICIENCY)

THE LICENSEE DID NOT COMPLY WITH COMMITMENTS MADE TO I&E IN LETTER C.N. DUNN TO B.H. GRIER DTD 11/29/79.
(7918 DEVIATION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

None.

FACILITY ITEMS (PLANS AND PROCEDURES):

The facility began an extended refueling/modification outage on 12/01/79 scheduled to last until late 7/80.

MANAGERIAL ITEMS:

A partial reorganization of station and corporate departments began on 12/16/79, and is expected to be fully implemented by 2/1/80.

PLANT STATUS:

Plant was operated at approximately 35% rated power through the reporting period and was shutdown on 11/30. The plant is presently in cold shutdown with preparations for refueling in progress.

LAST IE SITE INSPECTION DATE: 12/12/79

INSPECTION REPORT NO: 79-29

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-041/03L	10/04/79	11/01/79	30 Day LER	Inst drift caused false indication of control rod misalignment
79-042/03L	10/13/79	11/13/79	30 Day LER	Loop A feedwater flow channel 3 indication periodically spiking due to loose printed circuit card in instrument loop
79-043/03L	03/01/79	11/15/79	30 Day LER	Diesel fuel oil samples not analyzed within prescribed time
79-044/03L	10/18/79	11/16/79	30 Day LER	Control room emergency air bottle pressure below limit
79-047/01T	11/10/79	11/21/79	14 Day LER	Reactor coolant leakage detection radiation monitoring sub-systems inoperable
79-049/01T	11/04/79	11/16/79	14 Day LER	Failure to take required containment atmosphere samples with particulate detector out of service

SUMMARY

INSPECTION SUMMARY

Aug 20-24 (79-18): Routine, unannounced inspection by two region-based inspectors (onsite-54 hours and corporate offices-26 hours). Inspection areas (6): QA/QC Administration; design changes/modification program; design changes/modifications; records management; offsite support staff; followup on previously identified items. Noncompliances: Five (Failure to store records; failure to document 10 CFR 50.59 evaluations; failure to maintain "As-Built" drawings in station files; failure to update T.S. and procedure subsequent to a modification; Station Engineering performing Category I engineering). Deviation: One (Failure to comply with commitments).

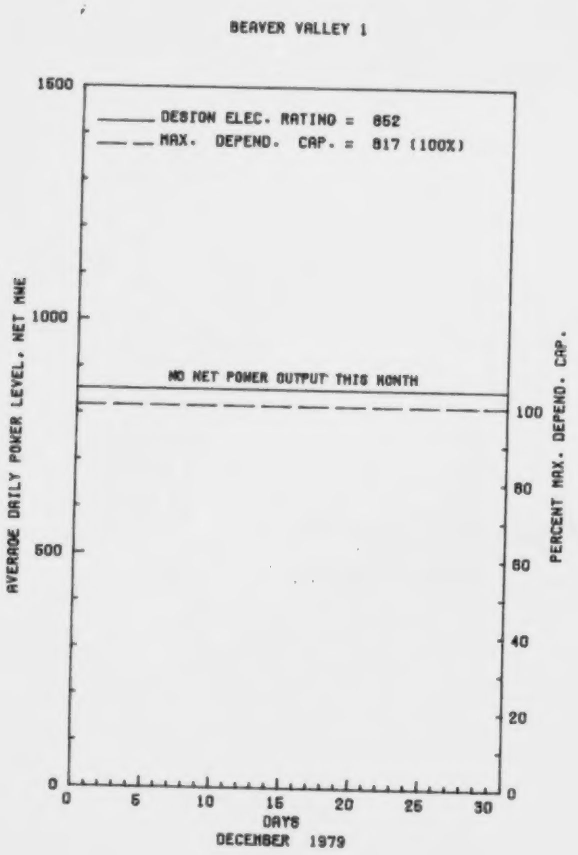
Oct 16-18 (79-23): Routine, onsite, unannounced inspection by one region-based inspector (31 hours). Inspection areas (3): General training; requalification training; craft personnel training. Noncompliances: One (Delinquent individual study).

REVIEWED BY OMPA LOEB: T. C. Cintula **DATE:** 2-11-80

REVIEWED BY NRR DOR: DLWigginton **DATE:** 2-13-80

BEAVER VALLEY 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-334
2. Utility Contact: A. E. Spitznogle (412) 643-5308
3. Licensed Thermal Power (MWt): 2652
4. Nameplate Rating (Gross MWe): 1026 x 0.9 = 923
5. Design Electrical Rating (Net MWe): 852
6. Maximum Dependable Capacity (Gross MWe): 845
7. Maximum Dependable Capacity (Net MWe): 817
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	8,760.0	32,160.0
12. Hours Reactor Was Critical	.0	3,578.6	13,774.7
13. Reactor Reserve Shutdown Hours	.0	.0	4,482.7
14. Hours Generator On-Line	.0	3,500.8	13,105.0
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	6,105,858	26,974,253
17. Gross Electrical Energy (MWH)	0	1,939,200	8,277,940
18. Net Electrical Energy (MWH)	0	1,778,375	7,570,743
19. Unit Service Factor	.0	40.0	40.7
20. Unit Availability Factor	.0	40.0	40.7
21. Unit Capacity Factor (MDC Net)	.0	24.8	28.8
22. Unit Capacity Factor (DER Net)	.0	23.8	27.6
23. Unit Forced Outage Rate	.0	56.3	45.0

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 07/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
18	12/01/79	5	744.0	C	1		ZZ	ZZZZZZ	UNIT SHUTDOWN FOR MAJOR MODIFICATIONS AND REFUELING OUTAGE. SCHEDULED STARTUP IN JULY, 1980.

SUMMARY

THE UNIT REMAINED SHUTDOWN THROUGHOUT THE REPORT PERIOD FOR A MAJOR MODIFICATION AND REFUELING OUTAGE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3- Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

Exhibit H:
Same source.

BIG ROCK POINT 1

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Michigan/Charlevoix 4 miles NE of Charlevoix
2.	Electric Reliability Council	ECAR
3.	Type of Reactor	BWR
4.	Power Level (MWT)	240
5.	Design Electrical Rating (MWE Net)	72
6.	Date of Initial Criticality	September 27, 1962
7.	Date of Commercial Operation	March 29, 1963
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10.	Licensee	Consumers Power Company
11.	Corporate Address	212 West Michigan Avenue Jackson, Michigan 49201
12.	Corporate Contact	R. B. DeWitt
13.	Architect/Engineer	Bechtel Corporation
14.	Nuclear Steam Supply System	GE
15.	Constructor	Bechtel Corporation

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Senior Inspector	D.C. Boyd
19.	IE Principal Inspector	K. R. Ridgway
20.	Licensing Project Manager	R. D. Silver
21.	Docket Number	50-155
22.	License Number/Date of Issue	DPR-6 issued 8/30/62
23.	Public Document Room	Charlevoix Public Library 107 Clinton Street Charlevoix, Michigan

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

- Currently operating at 208 Mwt. Plan to shutdown for one to two weeks late on December 31, 1979 to make modifications to comply with the requirements of NUREG 0578.

LAST IE SITE INSPECTION DATE: November 13-16, 1979

INSPECTION REPORT NO: 79-16

SUMMARY

INSPECTION SUMMARY

- Inspection on November 6-7 and 15-16, (79-14): Licensee actions relative to IE Bulletins No. 79-14, No. 79-04, and No. 79-02 including general discussion on NRC requirements, work procedure review, observation of work, and followup on previous identified items. The inspection involved 11 inspector hours onsite and at the licensee's A-E office by one NRC inspector. No items of noncompliance or deviations were identified.
- Inspection on October 29 through November 1, (79-15): Included review of: Security Plan; Security Program Audit; Security Organization; Physical Barriers (Protected area); Physical Barriers (Vital area); Lighting; Access control (Identification, Authorization, and Badging); Access control (Search); Access control (Escorting); Access control (Vital areas); Alarm stations; Detection aids; Assessment aids; Communications; Testing and maintenance; Compensatory measures; Power Supply; Response; Locks, keys and combinations; Records and reports; Follow up of IE Bulletin 79-16. The inspection involved 60 inspector hours onsite by two NRC inspectors and 15 inspector hours were during offshift periods. The licensee was found to be in compliance with NRC requirements within the 20 areas examined except for the following two items. Access Control (Search) - The licensee did not conduct searches in accordance with security plan commitmemts; and Locks, keys and combinations - A communications cabinet was inadequately locked.
- Inspection on November 13-16, (79-16): Routine, unannounced inspection of shutdown margin determination; core power distribution limits; control rod scram time tests; control rod sequence and reactivity checks; reactivity anomaly determination; review of specification field change. The inspection involved 22 inspector hours onsite by one NRC inspector. Of all the areas inspected, no items of noncompliance or deviations were identified.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

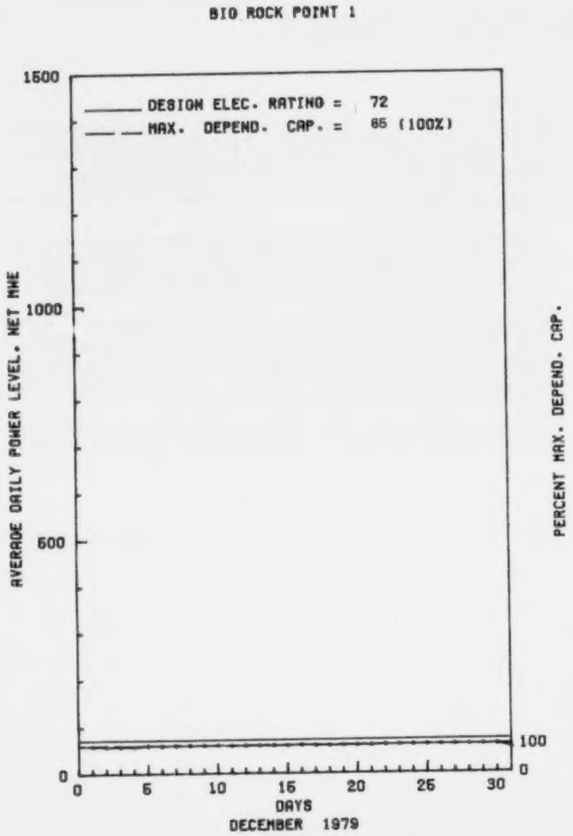
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-26/ 03L-0	11/01/79	12/03/79	30 Day LER	During monthly specific gravity checks on the Reactor Depressurizing System Battery C, Cell 48 read 1.194.
79-27/ 03L-0	11/15/79	12/11/79	30 Day LER	During power operation, the Emergency Condenser System temperatures indicated through seat leakage was occurring at the outlet valve, MO7053, of the No. 2 loop.

REVIEWED BY OMPA LOEB: Jack J. Rizzo DATE: 2-1-80

REVIEWED BY NRR DOR: DATE: 2-13-80

BIG ROCK POINT 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

ITEM 21 CALCULATED
BY USING A WEIGHTED

1. Docket Number: 50-155
2. Utility Contact: G. Himebauch (616) 547-6537 x180
3. Licensed Thermal Power (MWt): 240
4. Nameplate Rating (Gross MWe): 78.6 x 0.85 = 6
5. Design Electrical Rating (Net MWe): 72
6. Maximum Dependable Capacity (Gross MWe): 67
7. Maximum Dependable Capacity (Net MWe): 65
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
MDC TO 65 FOR FUEL THERMAL HYDRAULIC MARGINS
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	8,760.0	146,423.0
12. Hours Reactor Was Critical	742.3	2,106.3	100,133.0
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	742.2	2,061.9	97,998.9
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	156,209	396,380	18,117,931
17. Gross Electrical Energy (MWH)	47,415	120,865	5,735,918
18. Net Electrical Energy (MWH)	44,786	113,674	5,428,337
19. Unit Service Factor	99.8	23.5	66.7
20. Unit Availability Factor	99.8	23.5	66.7
21. Unit Capacity Factor (MDC Net)	95.1	20.6	54.5
22. Unit Capacity Factor (DER Net)	83.6	18.0	51.3
23. Unit Forced Outage Rate	.0	70.2	29.1

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 01/16/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
6	12/31/79	S	1.8	D	1		IB	INSTRU	REGULATORY SHUTDOWN TO IMPLEMENT REQUIREMENTS OF NUREG-0578. (RELIEF VALVE POSITION INDICATION, MANUAL RESET OF CONTAINMENT ISOLATION AND CORE DAMAGE ASSESSMENT RADIATION MONITORS).

SUMMARY

OPERATION WAS AT NEAR FULL POWER DURING THE REPORT PERIOD, WITH A REQUIRED SHUTDOWN ON 12/31/79 FOR CHECKING RELIEF VALVE POSITION INDICATION, MANUAL RESET OF CONTAINMENT ISOLATION, AND CORE DAMAGE ASSESSMENT RADIATION MONITORS.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

BROWNS FERRY 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Alabama/Limestone 10 miles NW of Decatur
2.	Electric Reliability Council	SERC
3.	Type of Reactor	BWR
4.	Power Level (MWT)	3293
5.	Design Electrical Rating (MWE Net)	1065
6.	Date of Initial Criticality	August 17, 1973
7.	Date of Commercial Operation	August 1, 1974
8.	Condenser Cooling Method	Once Through With Forced Draft Cooling Towers
9.	Condenser Cooling Water Source	Tennessee River

Utility & Contractor Information

10.	Licensee	Tennessee Valley Authority
11.	Corporate Address	500A Chestnut Street Tower II Chattanooga, Tennessee 37401
12.	Corporate Contact	H. G. Parris, Manager of Power.
13.	Architect/Engineer	TVA
14.	Nuclear Steam Supply System	GE
15.	Constructor	TVA

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. Lewis
18.	IE Senior Inspector	H. C. Dance
19.	IE Resident Inspector	R. Sullivan
20.	Licensing Project Manager	R. Clark
21.	Docket Number	50-259
22.	License Number/Date of Issue	DPR-33, 6/26/73
23.	Public Document Room	Athens Public Library South and Forrest Athens, Alabama 35611

INSPECTION STATUS

ENFORCEMENT STATUS

AS REQUIRED BY 10 CFR 50.59 AND EN DES EP 4.02, AN UNREVIEWED SAFETY QUESTION DETERMINATION (USQD) WILL BE MADE FOR ALL CHANGES TO THE FACILITY AS DESCRIBED IN THE SAFETY ANALYSIS REPORT. THE DETERMINATION SHALL ADDRESS THREE SPECIFIC ITEMS WHICH PROVIDE THE BASES FOR THE DETERMINATION THAT THE CHANGE DOES NOT INVOLVE AN UNREVIEWED SAFETY QUESTION. CONTRARY TO THE ABOVE, ENGINEERING CHANGE NOTICES P-0124, P-0081, L 2073 AND L 1967 WERE IMPLEMENTED WITH USQD'S WHICH DID NOT ADDRESS ALL THREE ITEMS NECESSARY TO ESTABLISH THAT THE DESIGN CHANGE DID NOT INVOLVE ANY UNREVIEWED SAFETY QUESTIONS.
(7930 INFRACTION)

AS REQUIRED BY 10 CFR 50, APPENDIX B, CRITERION III DESIGN CONTROL MEASURES SHALL PROVIDE FOR VERIFYING OR CHECKING THE ADEQUACY OF DESIGN, SUCH AS THE PERFORMANCE OF DESIGN REVIEWS. THE ACCEPTED QUALITY ASSURANCE PROGRAM COMMITS TVA TO ANSI N45.2.11-1974. ANSI N45.2.11, SECTION 3.1 STATES APPLICABLE DESIGN INPUTS, SUCH AS DESIGN BASES, REGULATORY REQUIREMENTS, CODES AND STANDARDS, SHALL BE IDENTIFIED, DOCUMENTED AND THEIR SELECTION REVIEWED AND APPROVED. DESIGN ANALYSES SHALL BE LEGIBLE AND BE IN A FORM SUITABLE FOR REPRODUCTION, FILING AND RETRIEVING. ADDITIONALLY, DESIGN DOCUMENTATION AND RECORDS WHICH PROVIDE EVIDENCE THAT THE DESIGN AND REVIEW PROCESS WAS PERFORMED IN ACCORDANCE WITH THE REQUIREMENTS OF THIS STANDARD SHALL BE COLLECTED, STORED AND MAINTAINED. CONTRARY TO THE ABOVE, 16 OF 16 SAFETY-RELATED DESIGN CHANGES REVIEWED SHOWED NO DOCUMENTARY EVIDENCE OF HAVING COMPLETED AND REVIEWED DESIGN INPUTS.
(7930 INFRACTION)

AS REQUIRED BY TECHNICAL SPECIFICATIONS 6.3.A.5 DETAILED WRITTEN PROCEDURES SHALL BE PREPARED, APPROVED AND ADHERED TO, FOR PREVENTIVE OR CORRECTIVE MAINTENANCE OPERATIONS WHICH COULD HAVE AN EFFECT ON THE SAFETY OF THE REACTOR. CONTRARY TO THE ABOVE, PROCEDURES WERE NOT BEING FOLLOWED FOR TROUBLE REPORTS 104531, 85353, 13512, 135958 AND 134173 A S FOLLOWS: (1) THESE WERE NOT BEING REVIEWED AS REQUIRED BY THE OQAM AND STANDARD PRACTICES BF 7.1; (2) THESE WERE NOT FILED AS REQUIRED BY STANDARD PRACTICES BAF8; AND (3) THESE WERE NOT DESIGNATED AS CRITICAL STRUCTURES, SYSTEMS AND COMPONENTS (CSSC) AS REQUIRED BY OQAM PART III, SECTION I, PARAGRAPH 4.2.
(7930 DEFICIENCY)

AS REQUIRED BY 10 CFR 50, APPENDIX B, CRITERION X, AN INSPECTION OF ACTIVITIES AFFECTING QUALITY SHALL BE PERFORMED BY INDIVIDUALS OTHER THAN THOSE WHO PERFORMED THE ACTIVITY. THE ACCEPTED QUALITY ASSURANCE PROGRAM, SECTION 17.2.10, REQUIRES INSPECTIONS TO BE PERFORMED DURING MAINTENANCE, MODIFICATIONS, OR REPAIR ACTIVITIES AFFECTING THE QUALITY OF CSSC ITEMS AT TVA PLANTS TO VERIFY CONFORMANCE WITH APPLICABLE REQUIREMENTS. CONTRARY TO THE ABOVE, UNIT 1 HPCI RUPTURE DISC WAS REPLACED AS REQUIRED BY TROUBLE REPORT 135958, WITHOUT THE NECESSARY INSPECTION REQUIREMENTS BEING SPECIFIED OR COMPLETED.
(7930 INFRACTION)

AS REQUIRED BY 10 CFR 50, APPENDIX B, CRITERION XI, A TEST PROGRAM SHALL BE ESTABLISHED WITH WRITTEN TEST PROCEDURES WHICH INCORPORATE THE REQUIREMENTS CONTAINED IN APPLICABLE DESIGN DOCUMENTS. THE ACCEPTED QUALITY ASSURANCE (QA) PROGRAM, SECTION 17.2.11, REQUIRES THAT MODIFICATIONS SHALL BE TESTED IN ACCORDANCE WITH ORIGINAL DESIGN REQUIREMENTS. THE UNREVIEWED SAFETY QUESTION DETERMINATION FOR ENGINEERING CHANGE NOTICE (ECN) L 1496, DISC REPLACEMENT OF FLOW CONTROL VALVES (FCV), STATED IN THE JUSTIFICATION FOR NO INCREASED POSSIBILITY OF ACCIDENT OR MALFUNCTION THAT THE CHATTERING FCV'S "WILL BE OPERATED OVER THEIR ENTIRE OPERATING RANGE AND OBSERVED BY QUALIFIED PERSONNEL...." CONTRARY TO THE ABOVE, RHR SERVICE WATER FCV'S 3-23-52, 3-23-34, 2-126, AND 3-23-40 WERE MODIFIED BUT THEY WERE NOT TESTED OVER THE ENTIRE OPERATING RANGE AND OBSERVED BY QUALIFIED PERSONNEL, AS REQUIRED BY THE UNREVIEWED SAFETY QUESTION DETERMINATION ANALYSIS.
(7930 INFRACTION)

AS REQUIRED BY 10 CFR 50, APPENDIX B, CRITERION XVII, "SUFFICIENT RECORDS SHALL BE MAINTAINED TO FURNISH EVIDENCE OF ACTIVITIES AFFECTING QUALITY...THE RECORDS SHALL ALSO INCLUDE CLOSELY RELATED DATA SUCH AS QUALIFICATIONS OF PERSONNEL...". THE ACCEPTED QUALITY ASSURANCE PROGRAM. SECTION 17.2.17 STATE IN PART, "THE PLANT SUPERINTENDENT SHALL PROVIDE STORAGE, PRESERVATION AND SAFEKEEPING OF THE REQUIRED QUALITY ASSURANCE RECORDS IN ACCORDANCE WITH TVA-ESTABLISHED REQUIREMENTS AND REGULATORY REQUIREMENTS...." CONTRARY TO THE ABOVE, CERTIFICATES OF TRAINING AND QUALIFICATION FOR INDIVIDUALS CONDUCTING RECEIVING INSPECTIONS WERE NOT BEING MAINTAINED BY THE PLANT QA STAFF. CONTRARY TO THE ABOVE, THE ACTUAL VOLTAGE READOUT WAS NOT PROPERLY ANNOTATED FOR ECN L 1911. CONTRARY TO THE ABOVE, MAINTENANCE INSTRUCTION MMI 14.4.1.3A (REQUIRED BY TROUBLE REPORT NUMBER 106568 WHICH WAS ISSUED FOR THE REPAIR OF A CRACKED SENSING LINE TO PT68-82) COULD NOT BE LOCATED. ADDITIONALLY, THE INSPECTION RECORDS FOR THIS ACTIVITY COULD NOT BE LOCATED.
(7930 DEFICIENCY)

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-032/ 01T-0	11/28/79	12/11/79	14 DAY LER	DURING INSPECTION OF RESTRAINTS PER IE BULLETIN 79-14 A HANGER WAS IDENTIFIED THAT WAS NOT ACCEPTABLE FOR ITS APPLICATION.

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

SCHEDULED FORTY DAY REFUELING OUTAGE TO BEGIN JANUARY 4, 1980.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

NINETY PERCENT FULL POWER OPERATION IN POWER COAST DOWN FOR END OF FUEL CYCLE.

LAST IE SITE INSPECTION DATE: DECEMBER 10-14, 1979

INSPECTION REPORT NO: 50-259/79-45

SUMMARY

INSPECTION SUMMARY

INSPECTION OCTOBER 22-27 (79-33): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 10 INSPECTOR-HOURS ON SITE IN THE AREAS OF CALIBRATION OF INPLANT TECHNICAL SPECIFICATION REQUIRED INSTRUMENTATION; CALIBRATION OF MEASURING AND TEST EQUIPMENT USED ON SAFETY RELATED SYSTEMS; SURVEILLANCE OF INPLANT TECHNICAL SPECIFICATION REQUIRED SYSTEMS (EQUIPMENT); AND REVIEW OF LICENSEE ACTION ON A PREVIOUS INSPECTION FINDING. OF THE FOUR AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION NOVEMBER 13-16 (79-40): AREAS INSPECTED INCLUDED REVIEW OF SECURITY ORGANIZATION, RESPONSE, SECURITY AUDIT, AND LICENSEE ACTION ON PREVIOUS INSPECTION FINDINGS. SINCE THE MANAGEMENT MEETING CONDUCTED JULY 24, 1979, INCREASED MANAGEMENT EMPHASIS HAS BEEN PLACED ON THE SECURITY PROGRAM THROUGH SECURITY AUDITS BY OFF-SITE TVA MANAGEMENT PERSONNEL, AND THE PLANT'S QUALITY ASSURANCE SECTION.

INSPECTION NOVEMBER 13-16 (79-41): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 19 INSPECTOR-HOURS ON SITE IN THE AREAS OF RADIOACTIVE WASTE TRANSPORTATION, BULLETIN FOLLOWUP, CIRCULAR FOLLOWUP, LICENSEE EVENT REPORT FOLLOWUP AND POSTING AND LABELLING INVOLVING POTENTIAL RADIATION HAZARDS. OF THE FIVE AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN FOUR AREAS, ONE APPARENT ITEM OF NONCOMPLIANCE WAS FOUND IN ONE AREA (DEFICIENCY - FAILURE TO ADEQUATELY LABEL CERTAIN CONTAINERS OF RADIOACTIVE MATERIALS).

INSPECTION NOVEMBER 27 (79-43): THIS INSPECTION CONSISTED OF NRC REGIONAL RESPONSE TO AND SUBSEQUENT OBSERVATION OF A DRILL INVOLVING A SIMULATED RELEASE OF RADIOACTIVE MATERIALS RESULTING FROM A TRANPORTATION ACCIDENT.

AS REQUIRED BY 10 CFR 50, APPENDIX B, CRITERION XVIII, "A COMPREHENSIVE SYSTEM OF PLANNED AND PERIODIC AUDITS SHALL BE CARRIED OUT TO VERIFY COMPLIANCE WITH ALL ASPECTS OF THE QUALITY ASSURANCE PROGRAM...". THE ACCEPTED QUALITY ASSURANCE PROGRAM, SECTION 17.2.18 STATES IN PART, "AUDITS SHALL BE CONDUCTED IN ACCORDANCE WITH A FORMAL AUDIT SCHEDULE WHICH SHALL BE UPDATED AT LEAST EVERY SIX MONTHS. EACH ELEMENT OF THE OFFICE OF POWER QUALITY ASSURANCE PROGRAM, SUCH AS DESIGN AND DOCUMENT CONTROL, AND EACH AREA OF PLANT OPERATIONS, SUCH AS NORMAL OPERATION, INSERVICE INSPECTIONS, AND RADIOLOGICAL CONTROL, SHALL BE AUDITED AT LEAST ONCE EVERY TWO YEARS." CONTRARY TO THE ABOVE, THE OUTAGE GROUP HAD NOT BEEN AUDITED DURING THE PERIOD FROM OCTOBER 6, 1977 TO OCTOBER 12, 1979, AND WAS NOT SCHEDULED FOR FUTURE AUDITS.
(7930 INFRACTION)

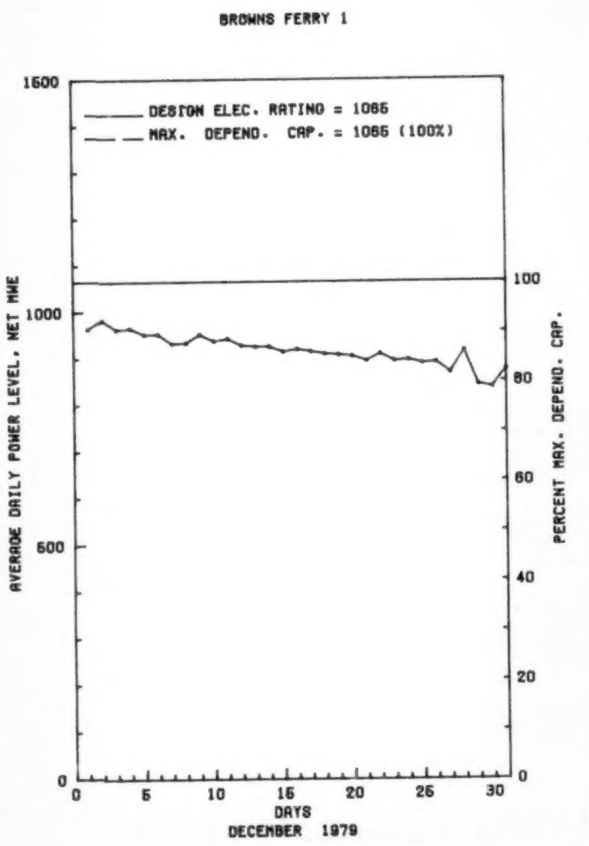
CONTRARY TO 10 CFR 20.203(F)(1) AND (2) ON NOVEMBER 13, 1979 THE LICENSEE HAD NOT LABELED, WITH THE APPROPRIATE WARNINGS AND INFORMATION SEVERAL CONTAINERS LOCATED IN THE UNIT 2 AND 3 TURBINE BUILDINGS.
(7941 DEFICIENCY)

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 2-11-80

REVIEWED BY NRR DOR: Dick Clark **DATE:** 02-13-80

BROWNS FERRY 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-259
2. Utility Contact: Don Green (205) 729-6846
3. Licensed Thermal Power (MWt): 3293
4. Nameplate Rating (Gross MWe): 1280 x 0.9 = 1152
5. Design Electrical Rating (Net MWe): 1065
6. Maximum Dependable Capacity (Gross MWe): 1098
7. Maximum Dependable Capacity (Net MWe): 1065
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	8,760.0	47,498.0
12. Hours Reactor Was Critical	744.0	8,045.6	28,277.3
13. Reactor Reserve Shutdown Hours	.0	290.4	4,689.5
14. Hours Generator On-Line	744.0	7,921.4	27,612.9
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	2,127,139	23,363,821	74,136,493
17. Gross Electrical Energy (MWH)	702,360	7,700,460	24,518,460
18. Net Electrical Energy (MWH)	683,103	7,495,748	23,801,968
19. Unit Service Factor	100.0	90.4	58.1
20. Unit Availability Factor	100.0	90.4	58.1
21. Unit Capacity Factor (MDC Net)	86.2	80.3	47.1
22. Unit Capacity Factor (DER Net)	86.2	80.3	47.1
23. Unit Forced Outage Rate	.0	4.4	33.0

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
REFUEL - JANUARY 1980.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	NONE								

SUMMARY

OPERATION WAS ROUTINE EOC DURING THE REPORT PERIOD, WITH BRIEF LOAD REDUCTIONS FOR TESTING.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H;
Same source.

BROWNS FERRY 2

FACILITY DATA

Facility Description

1. Location (state/countv) (nearest population center)	Alabama/Limestone 10 miles NW of Decatur
2. Electric Reliability Council	SERC
3. Type of Reactor	BWR
4. Power Level (MWT)	3293
5. Design Electrical Rating (MWE Net)	1065
6. Date of Initial Criticality	July 20, 1974
7. Date of Commercial Operation	March 1, 1975
8. Condenser Cooling Method	Once Through With Forced Draft Cooling Towers
9. Condenser Cooling Water Source	Tennessee River

Utility & Contractor Information

10. Licensee	Tennessee Valley Authority
11. Corporate Address	500A Chestnut Street Tower II Chattanooga, Tennessee 37401
12. Corporate Contact	H. G. Parris, Manager of Power
13. Architect/Engineer	TVA
14. Nuclear Steam Supply System	GE
15. Constructor	TVA

Regulatory Information

16. IE Region responsible	II
17. IE Branch Chief	R. C. Lewis
18. IE Senior Inspector	H. C. Dance
19. IE Resident Inspector	R. Sullivan
20. Licensing Project Manager	R. Clark
21. Docket Number	50-260
22. License Number/Date of Issue	DPR-52, 6/28/74
23. Public Document Room	Athens Public Library South and Forrest Athens, Alabama 35611

INSPECTION STATUS

ENFORCEMENT STATUS

AS REQUIRED BY 10 CFR 50.59 AND EN DES EP 4.02, AN UNREVIEWED SAFETY QUESTION DETERMINATION (USQD) WILL BE MADE FOR ALL CHANGES TO THE FACILITY AS DESCRIBED IN THE SAFETY ANALYSIS REPORT. THE DETERMINATION SHALL ADDRESS THREE SPECIFIC ITEMS WHICH PROVIDE THE BASES FOR THE DETERMINATION THAT THE CHANGE DOES NOT INVOLVE AN UNREVIEWED SAFETY QUESTION. CONTRARY TO THE ABOVE, ENGINEERING CHANGE NOTICES P-0124, P-0081, L 2073 AND L 1967 WERE IMPLEMENTED WITH USQD'S WHICH DID NOT ADDRESS ALL THREE ITEMS NECESSARY TO ESTABLISH THAT THE DESIGN CHANGE DID NOT INVOLVE ANY UNREVIEWED SAFETY QUESTIONS.
(7930 INFRACTION)

AS REQUIRED BY 10 CFR 50, APPENDIX B, CRITERION III DESIGN CONTROL MEASURES SHALL PROVIDE FOR VERIFYING OR CHECKING THE ADEQUACY OF DESIGN, SUCH AS THE PERFORMANCE OF DESIGN REVIEWS. THE ACCEPTED QUALITY ASSURANCE PROGRAM COMMITS TVA TO ANSI N45.2.11-1974. ANSI N45.2.11, SECTION 3.1 STATES APPLICABLE DESIGN INPUTS, SUCH AS DESIGN BASES, REGULATORY REQUIREMENTS, CODES AND STANDARDS, SHALL BE IDENTIFIED, DOCUMENTED AND THEIR SELECTION REVIEWED AND APPROVED. DESIGN ANALYSES SHALL BE LEGIBLE AND BE IN A FORM SUITABLE FOR REPRODUCTION, FILING AND RETRIEVING. ADDITIONALLY, DESIGN DOCUMENTATION AND RECORDS WHICH PROVIDE EVIDENCE THAT THE DESIGN AND REVIEW PROCESS WAS PERFORMED IN ACCORDANCE WITH THE REQUIREMENTS OF THIS STANDARD SHALL BE COLLECTED, STORED AND MAINTAINED. CONTRARY TO THE ABOVE, 16 OF 16 SAFETY-RELATED DESIGN CHANGES REVIEWED SHOWED NO DOCUMENTARY EVIDENCE OF HAVING COMPLETED AND REVIEWED DESIGN INPUTS.
(7930 INFRACTION)

AS REQUIRED BY TECHNICAL SPECIFICATIONS 6.3.A.5 DETAILED WRITTEN PROCEDURES SHALL BE PREPARED, APPROVED AND ADHERED TO, FOR PREVENTIVE OR CORRECTIVE MAINTENANCE OPERATIONS WHICH COULD HAVE AN EFFECT ON THE SAFETY OF THE REACTOR. CONTRARY TO THE ABOVE, PROCEDURES WERE NOT BEING FOLLOWED FOR TROUBLE REPORTS 104531, 85353, 13512, 135958 AND 134173 A S FOLLOWS: (1) THESE WERE NOT BEING REVIEWED AS REQUIRED BY THE OQAM AND STANDARD PRACTICES BF 7.1; (2) THESE WERE NOT FILED AS REQUIRED BY STANDARD PRACTICES BAF8; AND (3) THESE WERE NOT DESIGNATED AS CRITICAL STRUCTURES, SYSTEMS AND COMPONENTS (CSSC) AS REQUIRED BY OQAM PART III, SECTION I, PARAGRAPH 4.2.
(7930 DEFICIENCY)

AS REQUIRED BY 10 CFR 50, APPENDIX B, CRITERION XI, A TEST PROGRAM SHALL BE ESTABLISHED WITH WRITTEN TEST PROCEDURES WHICH INCORPORATE THE REQUIREMENTS CONTAINED IN APPLICABLE DESIGN DOCUMENTS. THE ACCEPTED QUALITY ASSURANCE (QA) PROGRAM, SECTION 17.2.11, REQUIRES THAT MODIFICATIONS SHALL BE TESTED IN ACCORDANCE WITH ORIGINAL DESIGN REQUIREMENTS. THE UNREVIEWED SAFETY QUESTION DETERMINATION FOR ENGINEERING CHANGE NOTICE (ECN) L 1496, DISC REPLACEMENT OF FLOW CONTROL VALVES (FCV), STATED IN THE JUSTIFICATION FOR NO INCREASED POSSIBILITY OF ACCIDENT OR MALFUNCTION THAT THE CHATTERING FCV'S "WILL BE OPERATED OVER THEIR ENTIRE OPERATING RANGE AND OBSERVED BY QUALIFIED PERSONNEL...". CONTRARY TO THE ABOVE, RHR SERVICE WATER FCV'S 3-23-52, 3-23-34, 2-126, AND 3-23-40 WERE MODIFIED BUT THEY WERE NOT TESTED OVER THE ENTIRE OPERATING RANGE AND OBSERVED BY QUALIFIED PERSONNEL, AS REQUIRED BY THE UNREVIEWED SAFETY QUESTION DETERMINATION ANALYSIS.
(7930 INFRACTION)

AS REQUIRED BY 10 CFR 50, APPENDIX B, CRITERION XVII, "SUFFICIENT RECORDS SHALL BE MAINTAINED TO FURNISH EVIDENCE OF ACTIVITIES AFFECTING QUALITY...THE RECORDS SHALL ALSO INCLUDE CLOSELY RELATED DATA SUCH AS QUALIFICATIONS OF PERSONNEL...". THE ACCEPTED QUALITY ASSURANCE PROGRAM, SECTION 17.2.17 STATE IN PART, "THE PLANT SUPERINTENDENT SHALL PROVIDE STORAGE, PRESERVATION AND SAFEKEEPING OF THE REQUIRED QUALITY ASSURANCE RECORDS IN ACCORDANCE WITH TVA-ESTABLISHED REQUIREMENTS AND REGULATORY REQUIREMENTS...". CONTRARY TO THE ABOVE, CERTIFICATES OF TRAINING AND QUALIFICATION FOR INDIVIDUALS CONDUCTING RECEIVING INSPECTIONS WERE NOT BEING MAINTAINED BY THE PLANT QA STAFF. CONTRARY TO THE ABOVE, THE ACTUAL VOLTAGE READOUT WAS NOT PROPERLY ANNOTATED FOR ECN L 1911. CONTRARY TO THE ABOVE, MAINTENANCE INSTRUCTION MMI 14.4.1.3A (REQUIRED BY TROUBLE REPORT NUMBER 106568 WHICH WAS ISSUED FOR THE REPAIR OF A CRACKED SENSING LINE TO PT68-82) COULD NOT BE LOCATED. ADDITIONALLY, THE INSPECTION RECORDS FOR THIS ACTIVITY COULD NOT BE LOCATED.
(7930 DEFICIENCY)

AS REQUIRED BY 10 CFR 50, APPENDIX B, CRITERION XVIII, "A COMPREHENSIVE SYSTEM OF PLANNED AND PERIODIC AUDITS SHALL BE CARRIED OUT TO VERIFY COMPLIANCE WITH ALL ASPECTS OF THE QUALITY ASSURANCE PROGRAM...". THE ACCEPTED QUALITY ASSURANCE PROGRAM, SECTION 17.2.18 STATES IN PART, "AUDITS SHALL BE CONDUCTED IN ACCORDANCE WITH A FORMAL AUDIT SCHEDULE WHICH SHALL BE UPDATED AT LEAST EVERY SIX MONTHS. EACH ELEMENT OF THE OFFICE OF POWER QUALITY ASSURANCE PROGRAM, SUCH AS DESIGN AND DOCUMENT CONTROL, AND EACH AREA OF PLANT OPERATIONS, SUCH AS NORMAL OPERATION, INSERVICE INSPECTIONS, AND RADIOLOGICAL CONTROL, SHALL BE AUDITED AT LEAST ONCE EVERY TWO YEARS." CONTRARY TO THE ABOVE, THE OUTAGE GROUP HAD NOT BEEN AUDITED DURING THE PERIOD FROM OCTOBER 6, 1977 TO OCTOBER 12, 1979, AND WAS NOT SCHEDULED FOR FUTURE AUDITS.
(7930 INFRACTION)

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
	NONE			

CONTRARY TO 10 CFR 20.203(F)(1) AND (2) ON NOVEMBER 13, 1979 THE LICENSEE HAD NOT LABELED, WITH THE APPROPRIATE WARNINGS AND INFORMATION SEVERAL CONTAINERS LOCATED IN THE UNIT 2 AND 3 TURBINE BUILDINGS.
(7941 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

ELECTRO-HYDRAULIC CONTROL SYSTEM PRESSURE REGULATOR MALFUNCTION. TROUBLE-SHOOTING IN PROGRESS.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

SEVENTY PERCENT OF FULL POWER OPERATION.

+ LAST IE SITE INSPECTION DATE: DECEMBER 10-14, 1979

+ INSPECTION REPORT NO: 50-260/79-45

SUMMARY

INSPECTION SUMMARY

+ INSPECTION OCTOBER 22-27 (79-33): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 10 INSPECTOR-HOURS ON SITE IN THE AREAS OF CALIBRATION OF INPLANT TECHNICAL SPECIFICATION REQUIRED INSTRUMENTATION; CALIBRATION OF MEASURING AND TEST EQUIPMENT USED ON SAFETY RELATED SYSTEMS; SURVEILLANCE OF INPLANT TECHNICAL SPECIFICATION REQUIRED SYSTEMS (EQUIPMENT); AND REVIEW OF LICENSEE ACTION ON A PREVIOUS INSPECTION FINDING. OF THE FOUR AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION NOVEMBER 13-16 (79-40): AREAS INSPECTED INCLUDED REVIEW OF SECURITY ORGANIZATION, RESPONSE, SECURITY AUDIT, AND LICENSEE ACTION ON PREVIOUS INSPECTION FINDINGS. SINCE THE MANAGEMENT MEETING CONDUCTED JULY 24, 1979, INCREASED MANAGEMENT EMPHASIS HAS BEEN PLACED ON THE SECURITY PROGRAM THROUGH SECURITY AUDITS BY OFF-SITE TVA MANAGEMENT PERSONNEL, AND THE PLANT'S QUALITY ASSURANCE SECTION.

INSPECTION NOVEMBER 13-16 (79-41): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 20 INSPECTOR-HOURS ON SITE IN THE AREAS OF RADIOACTIVE WASTE TRANSPORTATION, BULLETIN FOLLOWUP, CIRCULAR FOLLOWUP, LICENSEE EVENT REPORT FOLLOWUP AND POSTING AND LABELLING INVOLVING POTENTIAL RADIATION HAZARDS. OF THE FIVE AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN FOUR AREAS, ONE APPARENT ITEM OF NON-COMPLIANCE WAS FOUND IN ONE AREA (DEFICIENCY - FAILURE TO ADEQUATELY LABEL CERTAIN CONTAINERS OF RADIOACTIVE MATERIALS).

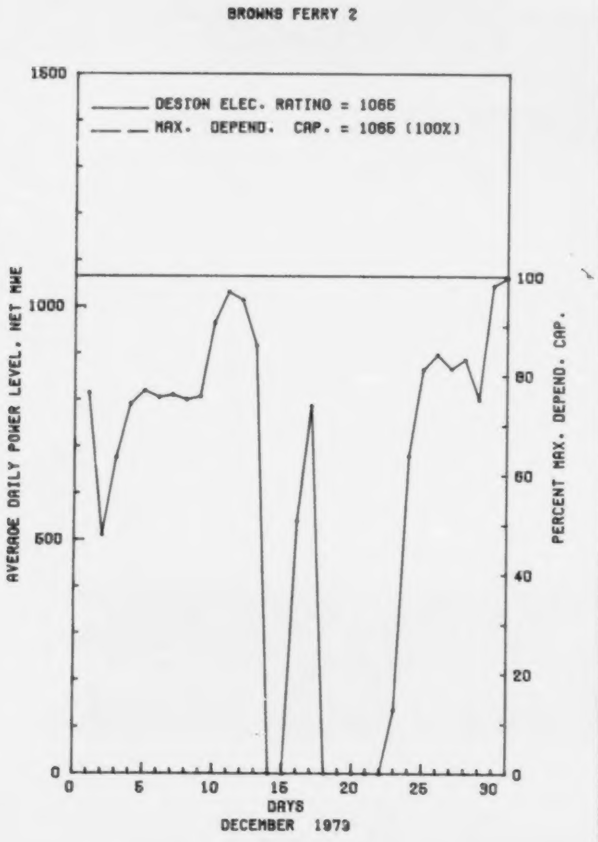
INSPECTION NOVEMBER 27 (79-43): THIS INSPECTION CONSISTED OF NRC REGIONAL RESPONSE TO AND SUBSEQUENT OBSERVATION OF A DRILL INVOLVING A SIMULATED RELEASE OF RADIOACTIVE MATERIALS RESULTING FROM A TRANPORTATION ACCIDENT. THE INSPECTION INVOLVED THREE INSPECTOR-HOURS BY TWO NRC INSPECTORS AND WAS BEGUN DURING THE REGULAR HOURS.

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 2-11-80

REVIEWED BY NRR DOR: Dick Clark **DATE:** 02-13-80

BROWNS FERRY 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-260
2. Utility Contact: Don Green (205) 729-6846
3. Licensed Thermal Power (MWt): 3293
4. Nameplate Rating (Gross MWe): 1280 x 0.9 = 1152
5. Design Electrical Rating (Net MWe): 1065
6. Maximum Dependable Capacity (Gross MWe): 1098
7. Maximum Dependable Capacity (Net MWe): 1065
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	8,760.0	42,409.0
12. Hours Reactor Was Critical	571.0	7,766.0	24,484.4
13. Reactor Reserve Shutdown Hours	173.0	298.1	11,774.9
14. Hours Generator On-Line	544.0	7,596.5	23,664.5
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,458,581	22,703,212	66,155,000
17. Gross Electrical Energy (MWH)	475,610	7,636,280	21,950,068
18. Net Electrical Energy (MWH)	461,952	7,441,305	21,331,469
19. Unit Service Factor	73.1	86.7	55.8
20. Unit Availability Factor	73.1	86.7	65.8
21. Unit Capacity Factor (MDC Net)	58.3	79.8	47.2
22. Unit Capacity Factor (DER Net)	58.3	79.8	47.2
23. Unit Forced Outage Rate	10.2	2.7	36.8

24. Shutdowns Scheduled Over Next 6 Months (Type,Date,and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
109	12/02/79	F	8.7	A	3				SUSPECTED EHC PRESSURE REGULATOR PRINTED CIRCUIT BOARD PROBLEMS, CAUSED CONTROL VALVE CLOSURE AND TURBINE TRIP.
110	12/13/79	F	53.4	B	2				MAINTENANCE TO "A" RECIRC PUMP AND FCV-1-5.
111	12/17/79	S	137.9	H	2				POST MODIFICATIONS TO THE PRIMARY CONTAINMENT ISOLATION SYSTEM.
112	12/28/79	S	.0	B	5				DERATED FOR CONTROL ROD SCRAM TESTING.

SUMMARY

OPERATION WAS ROUTINE DURING THE REPORT PERIOD, EXCEPT FOR A HIGH NEUTRON FLUX SCRAM ON 12/02/79 FROM ABRUPT MOVEMENT OF THE TURBINE CONTROL VALVES, AND MANUAL REACTOR SCRAMS ON 12/13/79 TO CHECK VIBRATION IN A RECIRCULATION MOTOR, PUMP, AND PIPING, AND ON 12/17/79 FOR MODIFICATIONS TO THE PRIMARY CONTAINMENT ISOLATION SYSTEM. A LOAD REDUCTION OCCURRED ON 12/28/79 FOR CONTROL ROD SCRAM TESTING.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

BROWNS FERRY 3

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Alabama/Limestone 10 miles NW of Decatur
2.	Electric Reliability Council	SERC
3.	Type of Reactor	BWR
4.	Power Level (MWT)	3293
5.	Design Electrical Rating (MWE Net)	1065
6.	Date of Initial Criticality	August 8, 1976
7.	Date of Commercial Operation	March 1, 1977
8.	Condenser Cooling Method	Once Through With Forced Draft Cooling Tower
9.	Condenser Cooling Water Source	Tennessee River

Utility & Contractor Information

10.	Licensee	Tennessee Valley Authority
11.	Corporate Address	500A Chestnut Street Tower II Chattanooga, Tennessee 37401
12.	Corporate Contact	H. G. Parris, Manager of Power
13.	Architect/Engineer	TVA
14.	Nuclear Steam Supply System	GE
15.	Constructor	TVA

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R.C. Lewis
18.	IE Senior Inspector	H. C. Dance
19.	IE Resident Inspector	R. Sullivan
20.	Licensing Project Manager	R. Clark
21.	Docket Number	50-296
22.	License Number/Date of Issue	DPR-68, 7/2/76
23.	Public Document Room	Athens Public Library South and Forrest Athens, Alabama 35611

INSPECTION STATUS

ENFORCEMENT STATUS

AS REQUIRED BY 10 CFR 50.59 AND EN DES EP 4.02, AN UNREVIEWED SAFETY QUESTION DETERMINATION (USQD) WILL BE MADE FOR ALL CHANGES TO THE FACILITY AS DESCRIBED IN THE SAFETY ANALYSIS REPORT. THE DETERMINATION SHALL ADDRESS THREE SPECIFIC ITEMS WHICH PROVIDE THE BASES FOR THE DETERMINATION THAT THE CHANGE DOES NOT INVOLVE AN UNREVIEWED SAFETY QUESTION. CONTRARY TO THE ABOVE, ENGINEERING CHANGE NOTICES P-0124, P-0081, L 2073 AND L 1967 WERE IMPLEMENTED WITH USQD'S WHICH DID NOT ADDRESS ALL THREE ITEMS NECESSARY TO ESTABLISH THAT THE DESIGN CHANGE DID NOT INVOLVE ANY UNREVIEWED SAFETY QUESTIONS.
(7930 INFRACTION)

AS REQUIRED BY 10 CFR 50, APPENDIX B, CRITERION III DESIGN CONTROL MEASURES SHALL PROVIDE FOR VERIFYING OR CHECKING THE ADEQUACY OF DESIGN, SUCH AS THE PERFORMANCE OF DESIGN REVIEWS. THE ACCEPTED QUALITY ASSURANCE PROGRAM COMMITS TVA TO ANSI N45.2.11-1974. ANSI N45.2.11, SECTION 3.1 STATES APPLICABLE DESIGN INPUTS, SUCH AS DESIGN BASES, REGULATORY REQUIREMENTS, CODES AND STANDARDS, SHALL BE IDENTIFIED, DOCUMENTED AND THEIR SELECTION REVIEWED AND APPROVED. DESIGN ANALYSES SHALL BE LEGIBLE AND BE IN A FORM SUITABLE FOR REPRODUCTION, FILING AND RETRIEVING. ADDITIONALLY, DESIGN DOCUMENTATION AND RECORDS WHICH PROVIDE EVIDENCE THAT THE DESIGN AND REVIEW PROCESS WAS PERFORMED IN ACCORDANCE WITH THE REQUIREMENTS OF THIS STANDARD SHALL BE COLLECTED, STORED AND MAINTAINED. CONTRARY TO THE ABOVE, 16 OF 16 SAFETY-RELATED DESIGN CHANGES REVIEWED SHOWED NO DOCUMENTARY EVIDENCE OF HAVING COMPLETED AND REVIEWED DESIGN INPUTS.
(7930 INFRACTION)

AS REQUIRED BY TECHNICAL SPECIFICATIONS 6.3.A.5 DETAILED WRITTEN PROCEDURES SHALL BE PREPARED, APPROVED AND ADHERED TO, FOR PREVENTIVE OR CORRECTIVE MAINTENANCE OPERATIONS WHICH COULD HAVE AN EFFECT ON THE SAFETY OF THE REACTOR. CONTRARY TO THE ABOVE, PROCEDURES WERE NOT BEING FOLLOWED FOR TROUBLE REPORTS 104531, 85353, 13512, 135958 AND 134173 A S FOLLOWS: (1) THESE WERE NOT BEING REVIEWED AS REQUIRED BY THE OQAM AND STANDARD PRACTICES BF 7.1; (2) THESE WERE NOT FILED AS REQUIRED BY STANDARD PRACTICES BAF8; AND (3) THESE WERE NOT DESIGNATED AS CRITICAL STRUCTURES, SYSTEMS AND COMPONENTS (CSSC) AS REQUIRED BY OQAM PART III, SECTION I, PARAGRAPH 4.2.
(7930 DEFICIENCY)

AS REQUIRED BY 10 CFR 50, APPENDIX B, CRITERION XI, A TEST PROGRAM SHALL BE ESTABLISHED WITH WRITTEN TEST PROCEDURES WHICH INCORPORATE THE REQUIREMENTS CONTAINED IN APPLICABLE DESIGN DOCUMENTS. THE ACCEPTED QUALITY ASSURANCE (QA) PROGRAM, SECTION 17.2.11, REQUIRES THAT MODIFICATIONS SHALL BE TESTED IN ACCORDANCE WITH ORIGINAL DESIGN REQUIREMENTS. THE UNREVIEWED SAFETY QUESTION DETERMINATION FOR ENGINEERING CHANGE NOTICE (ECN) L 1496, DISC REPLACEMENT OF FLOW CONTROL VALVES (FCV), STATED IN THE JUSTIFICATION FOR NO INCREASED POSSIBILITY OF ACCIDENT OR MALFUNCTION THAT THE CHATTERING FCV'S "WILL BE OPERATED OVER THEIR ENTIRE OPERATING RANGE AND OBSERVED BY QUALIFIED PERSONNEL...." CONTRARY TO THE ABOVE, RHR SERVICE WATER FCV'S 3-23-52, 3-23-34, 2-126, AND 3-23-40 WERE MODIFIED BUT THEY WERE NOT TESTED OVER THE ENTIRE OPERATING RANGE AND OBSERVED BY QUALIFIED PERSONNEL, AS REQUIRED BY THE UNREVIEWED SAFETY QUESTION DETERMINATION ANALYSIS.
(7930 INFRACTION)

AS REQUIRED BY 10 CFR 50, APPENDIX B, CRITERION XVII, "SUFFICIENT RECORDS SHALL BE MAINTAINED TO FURNISH EVIDENCE OF ACTIVITIES AFFECTING QUALITY...THE RECORDS SHALL ALSO INCLUDE CLOSELY RELATED DATA SUCH AS QUALIFICATIONS OF PERSONNEL...". THE ACCEPTED QUALITY ASSURANCE PROGRAM, SECTION 17.2.17 STATE IN PART, "THE PLANT SUPERINTENDENT SHALL PROVIDE STORAGE, PRESERVATION AND SAFEKEEPING OF THE REQUIRED QUALITY ASSURANCE RECORDS IN ACCORDANCE WITH TVA-ESTABLISHED REQUIREMENTS AND REGULATORY REQUIREMENTS...". CONTRARY TO THE ABOVE, CERTIFICATES OF TRAINING AND QUALIFICATION FOR INDIVIDUALS CONDUCTING RECEIVING INSPECTIONS WERE NOT BEING MAINTAINED BY THE PLANT QA STAFF. CONTRARY TO THE ABOVE, THE ACTUAL VOLTAGE READOUT WAS NOT PROPERLY ANNOTATED FOR ECN L 1911. CONTRARY TO THE ABOVE, MAINTENANCE INSTRUCTION MMI 14.4.1.3A (REQUIRED BY TROUBLE REPORT NUMBER 106568 WHICH WAS ISSUED FOR THE REPAIR OF A CRACKED SENSING LINE TO PT68-82) COULD NOT BE LOCATED. ADDITIONALLY, THE INSPECTION RECORDS FOR THIS ACTIVITY COULD NOT BE LOCATED.
(7930 DEFICIENCY)

AS REQUIRED BY 10 CFR 50, APPENDIX B, CRITERION XVIII, "A COMPREHENSIVE SYSTEM OF PLANNED AND PERIODIC AUDITS SHALL BE CARRIED OUT TO VERIFY COMPLIANCE WITH ALL ASPECTS OF THE QUALITY ASSURANCE PROGRAM...." THE ACCEPTED QUALITY ASSURANCE PROGRAM, SECTION 17.2.18 STATES IN PART, "AUDITS SHALL BE CONDUCTED IN ACCORDANCE WITH A FORMAL AUDIT SCHEDULE WHICH SHALL BE UPDATED AT LEAST EVERY SIX MONTHS. EACH ELEMENT OF THE OFFICE OF POWER QUALITY ASSURANCE PROGRAM, SUCH AS DESIGN AND DOCUMENT CONTROL, AND EACH AREA OF PLANT OPERATIONS, SUCH AS NORMAL OPERATION, INSERVICE INSPECTIONS, AND RADIOLOGICAL CONTROL, SHALL BE AUDITED AT LEAST ONCE EVERY TWO YEARS." CONTRARY TO THE ABOVE, THE OUTAGE GROUP HAD NOT BEEN AUDITED DURING THE PERIOD FROM OCTOBER 6, 1977 TO OCTOBER 12, 1979, AND WAS NOT SCHEDULED FOR FUTURE AUDITS.
(7930 INFRACTION)

AS REQUIRED BY 10 CFR 50, APPENDIX B, CRITERION V, INSTRUCTION PROCEDURES AND DRAWINGS, "ACTIVITIES AFFECTING QUALITY SHALL BE PRESCRIBED BY DOCUMENTED INSTRUCTIONS ... AND SHALL BE ACCOMPLISHED IN ACCORDANCE WITH THESE INSTRUCTIONS...". THE ACCEPTED QUALITY ASSURANCE PROGRAM, TVA-TR75-1A, REV. 3, STATES IN PART, "MEASURING AND TEST EQUIPMENT UTILIZED IN OR RELATED TO OPERATION OF TVA'S NUCLEAR PLANTS SHALL BE CONTROLLED IN ACCORDANCE WITH WRITTEN PROCEDURES OR INSTRUCTIONS." BROWNS FERRY OPERATIONAL QUALITY ASSURANCE MANUAL 3.4.1. "MEASURING AND TEST EQUIPMENT WHOSE CALIBRATION DUE DATE HAS PASSED ... SHALL BE CLEARLY TAGGED TO INDICATE ITS QUALITY STATUS, REMOVED FROM SERVICE, AND SEGREGATED FROM ACCEPTABLE EQUIPMENT. CONTRARY TO THE ABOVE, ACTICITIES AFFECTING QUALITY WERE NOT ACCOMPLISHED IN ACCORDANCE WITH THE INSTRUCTIONS IN THAT CURRENT CALIBRATOR C-4, WAS NOT TAGGED NOR SEGREGATED FROM ACCEPTABLE EQUIPMENT ON 10/24/79. FURTHER THE EQUIPMENT HAD BEEN USED ELEVEN TIMES TO CALIBRATE SAFETY RELATED INSTRUMENTS IN UNIT 3 SINCE THE LAST CALIBRATED DUE DATE OF 8/6/79.
(7933 INFRACTION)

TECHNICAL SPECIFICATION 3.10.A.1 REQUIRES THAT THE REACTOR MODE SWITCH SHALL BE LOCKED IN THE 'REFUEL' POSITION DURING CORE ALTERATIONS. CONTRARY TO THE ABOVE, DURING CORE ALTERATIONS ON NOVEMBER 9, 1979 THE REACTOR MODE SWITCH WAS OBSERVED TO BE IN THE REFUEL POSITION BUT THE KEY WAS LEFT INSERTED IN THE LOCK, THUS ALLOWING THE SWITCH THE CAPABILITY OF BEING READILY MANIPULATED.
(7936 INFRACTION)

CONTRARY TO 10 CFR 20.203(F)(1) AND (2) ON NOVEMBER 13, 1979 THE LICENSEE HAD NOT LABELED, WITH THE APPROPRIATE WARNINGS AND INFORMATION SEVERAL CONTAINERS LOCATED IN THE UNIT 2 AND 3 TURBINE BUILDINGS.
(7941 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

EXPECTED COMPLETION OF REFUELING OUTAGE IS DECEMBER 5, 1979.

LAST IE SITE INSPECTION DATE: DECEMBER 10-14, 1979

INSPECTION REPORT NO: 50-296/79-45

SUMMARY

INSPECTION SUMMARY

INSPECTION OCTOBER 22-27 (79-33): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 11 INSPECTOR-HOURS ON SITE IN THE AREAS OF CALIBRATION OF INPLANT TECHNICAL SPECIFICATION REQUIRED INSTRUMENTATION; CALIBRATION OF MEASURING AND TEST EQUIPMENT USED ON SAFETY RELATED SYSTEMS; SURVEILLANCE OF INPLANT TECHNICAL SPECIFICATION REQUIRED SYSTEMS (EQUIPMENT); AND REVIEW OF LICENSEE ACTION ON A PREVIOUS INSPECTION FINDING. OF THE FOUR AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION NOVEMBER 13-16 (79-40): AREAS INSPECTED INCLUDED REVIEW OF SECURITY ORGANIZATION, RESPONSE, SECURITY AUDIT, AND LICENSEE ACTION ON PREVIOUS INSPECTION FINDINGS. SINCE THE MANAGEMENT MEETING CONDUCTED JULY 24, 1979, INCREASED MANAGEMENT EMPHASIS HAS BEEN PLACED ON THE SECURITY PROGRAM THROUGH SECURITY AUDITS BY OFF-SITE TVA MANAGEMENT PERSONNEL, AND THE PLANT'S QUALITY ASSURANCE SECTION.

INSPECTION NOVEMBER 13-16 (79-41): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 19 INSPECTOR-HOURS ON SITE IN THE AREAS OF RADIOACTIVE WASTE TRANSPORTATION, BULLETIN FOLLOWUP, CIRCULAR FOLLOWUP, LICENSEE EVENT REPORT FOLLOWUP AND POSTING AND LABELLING INVOLVING POTENTIAL RADIATION HAZARDS. OF THE FIVE AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN FOUR AREAS, ONE APPARENT ITEM OF NON-COMPLIANCE WAS FOUND IN ONE AREA (DEFICIENCY - FAILURE TO ADEQUATELY LABEL CERTAIN CONTAINERS OF RADIOACTIVE MATERIALS).

INSPECTION NOVEMBER 27 (79-43): THIS INSPECTION CONSISTED OF NRC REGIONAL RESPONSE TO AND SUBSEQUENT OBSERVATION OF A DRILL INVOLVING A SIMULATED RELEASE OF RADIOACTIVE MATERIALS RESULTING FROM A TRANPORTATION ACCIDENT. THE INSPECTION INVOLVED THREE INSPECTOR-HOURS BY TWO NRC INSPECTORS AND

REPORTS RECEIVED FROM LICENSEE

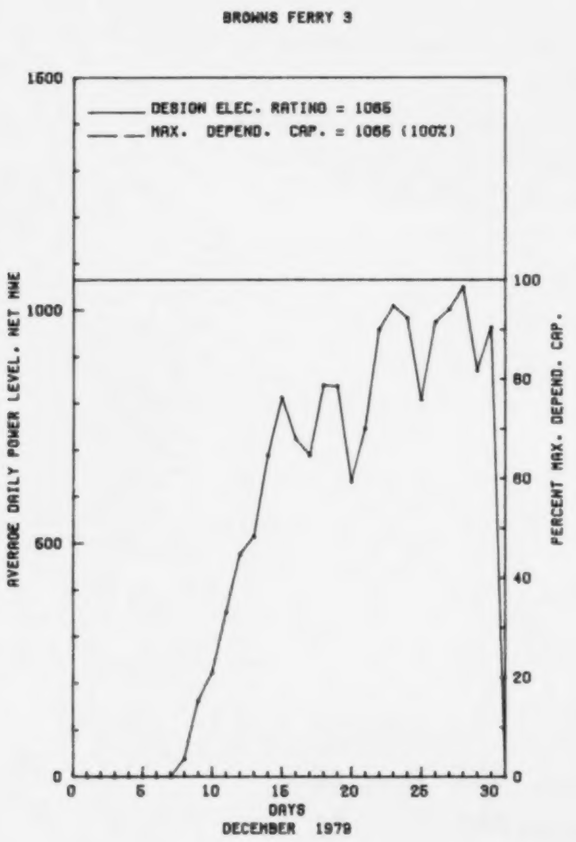
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-019/ 01T-0	12/04/79	12/17/79	14 DAY LER	CRACK FOUND IN CORE SPRAY PIPING WHICH HAD BEEN REMOVED FOR REPLACEMENT.

REVIEWED BY OMPA LOEB: John J. **DATE:** 2-1-80

REVIEWED BY NRR DOR: Dick Clark **DATE:** 02-13-80

BROWNS FERRY 3

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-296
2. Utility Contact: Don Green (205) 729-6846
3. Licensed Thermal Power (MWt): 3293
4. Nameplate Rating (Gross MWe): 1280 x 0.9 = 1152
5. Design Electrical Rating (Net MWe): 1065
6. Maximum Dependable Capacity (Gross MWe): 1098
7. Maximum Dependable Capacity (Net MWe): 1065
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	8,760.0	24,864.0
12. Hours Reactor Was Critical	591.7	5,816.3	18,907.7
13. Reactor Reserve Shutdown Hours	131.5	552.9	1,658.4
14. Hours Generator On-Line	536.3	5,708.4	18,438.4
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,231,495	16,956,484	53,127,866
17. Gross Electrical Energy (MWH)	403,790	5,635,080	17,402,580
18. Net Electrical Energy (MWH)	390,613	5,482,585	16,887,773
19. Unit Service Factor	71.8	65.2	74.2
20. Unit Availability Factor	71.8	65.2	74.2
21. Unit Capacity Factor (MDC Net)	49.3	58.8	63.8
22. Unit Capacity Factor (DER Net)	49.3	58.8	63.8
23. Unit Forced Outage Rate	4.6	8.0	9.0

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: mid-January

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
109	08/24/79	S	184.2	C	4				REFUELING
110	12/20/79	S	.0	H	5				DERATED FOR CONTROL ROD PATTERN ADJUSTMENT.
111	12/24/79	S	.0	H	5				DERATED FOR CONTROL ROD SEQUENCE EXCHANGE.
112	12/30/79	F	25.5	A	2				GROUND IN AUXILIARY POWER SUPPLY CABLES FROM COOLING TOWERS TO BUS TIE BOARDS. INSTALLATION OF OVERHEAD CABLES IN PROGRESS.

SUMMARY

THE UNIT STARTED UP FROM ITS REFUELING OUTAGE ON 12/07/79, AND OPERATED ROUTINELY WITH SEVERAL REDUCTIONS FOR MAINTENANCE AND CONTROL ROD ADJUSTMENT. ON 12/30/79, THE REACTOR WAS MANUALLY SCRAMMED TO INSTALL OVERHEAD CABLES FROM THE COOLING TOWER SWITCH GEAR TO THE BUS TIE BOARDS.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

BRUNSWICK 1

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	North Carolina/Brunswick 3 miles N of Southport
2.	Electric Reliability Council	Southeastern
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2436
5.	Design Electrical Rating (MWE Net)	821
6.	Date of Initial Criticality	October 8, 1976
7.	Date of Commercial Operation	March 18, 1977
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Cape Fear River

Utility & Contractor Information

10.	Licensee	Carolina Power and Light Company
11.	Corporate Address	411 Fayetteville Street Raleigh, North Carolina 27602
12.	Corporate Contact	J. A. Jones Executive Vice President
13.	Architect/Engineer	United Engineers & Constructors, Inc.
14.	Nuclear Steam Supply System	GE
15.	Constructor	Brown and Root, Inc.

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R.C. Lewis
18.	IE Senior Inspector	P.J. Kellogg
19.	IE Resident Inspector	J. E. Ouzts
20.	Licensing Project Manager	J. Hannon
21.	Docket Number	50-325
22.	License Number/Date of Issue	DPR-71, 9/8/76
23.	Public Document Room	Southport-Brunswick County Library 109 W. Moore Street Southport, North Carolina 28461

INSPECTION STATUS

ENFORCEMENT STATUS

AS REQUIRED BY SECTION 3.7.2.2 OF THE TECHNICAL SPECIFICATIONS, WITH ONE OR MORE OF THE DIESEL GENERATOR SPRAY AND/OR SPRINKLER SYSTEMS INOPERABLE, ESTABLISH A CONTINUOUS FIRE WATCH WITH BACKUP FIRE SUPPRESSION EQUIPMENT WITHIN 1 HOUR. CONTRARY TO THE ABOVE ON OCTOBER 16 THE INSPECTOR FOUND FIRE PROTECTION SPRINKLER AND STANDPIPE SYSTEMS ISOLATED TO DIESEL GENERATOR NO 3. THE ISOLATED SYSTEMS WERE INDICATED BY LOW PRESSURE ANNUNCIATORS FOR THE SYSTEMS. A FOLLOWUP INVESTIGATION REVEALED THAT TWO DELUGE VALVES (DV-15 AND DV-17) HAD BEEN ISOLATED FOR MAINTENANCE BY MANUALLY CLOSING THREE ISOLATION VALVES. THIS ACTION WAS TAKEN WITHOUT NOTIFYING OPERATING PERSONNEL, AND HAVING THE SYSTEMS PLACED IN A LIMITING CONDITION FOR OPERATION (LCO) AND TAKING COMPENSATORY MEASURES REQUIRED BY THE TECHNICAL SPECIFICATIONS. AN LCO WAS ESTABLISHED AFTER THE INSPECTOR FOUND THE CONDITION, NOTIFIED OPERATING PERSONNEL AND THEY HAD INVESTIGATED THE CAUSE OF THE ANNUNICATOR ALARM.
(7939 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

NORMAL FULL POWER OPERATION.

LAST IE SITE INSPECTION DATE: OCTOBER 27 - NOVEMBER 30, 1979

INSPECTION REPORT NO: 50-325/79-43

SUMMARY

INSPECTION SUMMARY

INSPECTION OCTOBER 19-21 (79-38): THIS SPECIAL, ANNOUNCED INSPECTION INVOLVED 86 INSPECTOR-HOURS ON SITE IN THE AREAS OF A REACTOR TRIP ON OCTOBER 19, 1979. OF THE AREA INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION SEPTEMBER 29 - OCTOBER 26 (79-39): THIS ROUTINE INSPECTION BY THE RESIDENT INSPECTOR INVOLVED 27 MANHOURS OF INSPECTION TIME ON SITE OF PREVIOUSLY IDENTIFIED EVENTS, PLANT OPERATIONS, RADIATION PROTECTION, PHYSICAL PROTECTION, QUALITY ASSURANCE, ORGANIZATION AND ADMINISTRATION AND UNIT 1 MAINTENANCE. OF THE SIX AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN FIVE AREAS. ONE APPARENT ITEM OF NONCOMPLIANCE WAS IDENTIFIED IN ONE AREA - FAILURE TO RECOGNIZE THAT A LIMITING CONDITION FOR OPERATION WAS ESTABLISHED AND TAKE COMPENSATORY MEASURES REQUIRED BY TECHNICAL SPECIFICATIONS, WHEN FIRE PROTECTION DELUGE VALVES WERE MANUALLY ISOLATED, CAUSING ISOLATION OF SECTIONS OF THE SPRINKLER AND STANDPIPE SYSTEMS TO DIESEL GENERATOR NO. 3.

INSPECTION NOVEMBER 15-16 (79-40): THIS SPECIAL, ANNOUNCED INSPECTION INVOLVED 18 INSPECTOR-HOURS ON SITE IN THE AREAS OF SEISMIC ANALYSIS FOR AS-BUILT SAFETY-RELATED PIPING SYSTEMS (IE BULLETIN 79-14). NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION OCTOBER 27 - NOVEMBER 30 (79-43): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 59 INSPECTOR-HOURS ON SITE IN THE AREAS OF PLANT OPERATIONS, PLANT TOURS, RADIATION PROTECTION, PHYSICAL PROTECTION, QUALITY ASSURANCE, ORGANIZATION AND ADMINISTRATION AND FOLLOWUP AND COVERAGE OF UNIT NO. 1 TRIPS. OF THE SEVEN AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

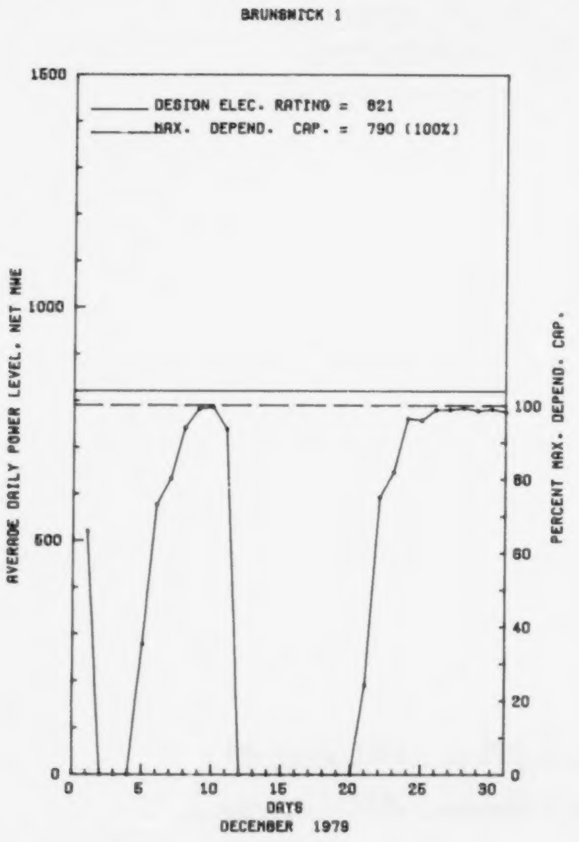
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-086/03L-0	11/19/79	12/11/79	30 DAY LER	CONTAINMENT ATMOSPHERIC MONITOR, CAC-AQH-1263 FAILED DOWNSCALE ON THE OXYGEN CHANNEL.
79-087/03L-0	10/25/79	11/16/79	30 DAY LER	TORUS WATER TEMPERATURE EXCEEDED LIMIT.
79-094/03L-0	11/16/79	12/14/79	30 DAY LER	DRIVE BELT BROKE ON CONTAINMENT ATMOSPHERIC MONITOR CAC-AQAH-1259.
79-097/03L-0	11/08/79	12/06/79	30 DAY LER	LOSS OF DG NO. 1 DURING A TEST.
79-099/03L-0	11/15/79	12/07/79	30 DAY LER	DEFECTIVE FULL-IN INDICATING LIGHTS ON CONTROL RODS 18-11 AND 26-11 AND DEFECTIVE FULLOUT LIGHT ON CONTROL ROD 18-31.
79-107/03L-0	11/20/79	12/12/79	30 DAY LER	SNUBBERS DAMAGED ON MAIN STEAM SAFETY RELIEF VALVE B21-F013H.

REVIEWED BY OMPA LOEB: John J. Rizzo DATE: 2-11-80

REVIEWED BY NRR DOR: John N. Hannon DATE: 2-13-80

BRUNSWICK 1

AVERAGE POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-325
2. Utility Contact: E. Willis (919) 457-9521
3. Licensed Thermal Power (MWt): 2436
4. Nameplate Rating (Gross MWe): 963 x 0.9 = 867
5. Design Electrical Rating (Net MWe): 821
6. Maximum Dependable Capacity (Gross MWe): 815
7. Maximum Dependable Capacity (Net MWe): 790
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	8,760.0	24,457.0
12. Hours Reactor Was Critical	492.3	5,079.0	17,389.6
13. Reactor Reserve Shutdown Hours	.0	.0	1,647.1
14. Hours Generator On-Line	447.8	4,783.0	16,332.8
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	939,301	9,882,306	33,829,496
17. Gross Electrical Energy (MWH)	314,614	3,307,542	11,238,819
18. Net Electrical Energy (MWH)	301,370	3,169,212	10,807,310
19. Unit Service Factor	60.2	54.6	66.8
20. Unit Availability Factor	60.2	54.6	66.8
21. Unit Capacity Factor (MDC Net)	51.3	45.8	55.9
22. Unit Capacity Factor (DER Net)	49.3	44.1	53.8
23. Unit Forced Outage Rate	15.3	16.1	21.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUELING - 04/22/80 - 1752 HOURS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
023	12/01/79	F	79.1	A	2		CB	VALVEX	UNIT WAS SEPARATED FROM GRID DUE TO HIGH DRYWELL LEAKAGE FROM REACTOR RECIRCULATION SUCTION & DISCHARGE VALVES LEAK-OFFS. SUCTION & DISCHARGE VALVES WERE LEAKING EXCESSIVELY BY STEM PACKING. PACKING ADJUSTED & VALVE GLANDS REPACKED AS NEEDED TO STOP LEAKAGE. A SOLENOID FOR THE STAGING FLOW VALVE WAS FOUND DEFECTIVE.
024	12/12/79	S	215.0	D	2		SH	VALVEX	THE UNIT WAS SEPARATED FROM THE GRID TO PERFORM A PLANT MODIFICATION TO THE SAFETY/RELIEF VALVES AND A SCHEDULED PIPE SHUBBER INSPECTION. THE OUTAGE WAS EXTENDED DUE TO PROBLEMS WITH REBUILDING MECHANICAL SEALS ON THE 1B REACTOR RECIRCULATION PUMP.
025	12/21/79	F	2.1	B	1		SH	VALVEX	THE UNIT WAS PLACED ON THE GRID THEN SEPARATED FROM THE GRID FOR CONSTRUCTION TO MAKE REPAIRS TO THE SAFETY RELIEF VALVE MODIFICATION **(SEE OUTAGE NO. 79-024)**.

SUMMARY

THE UNIT EXPERIENCED MAJOR OUTAGE ON 12/01/79 DUE TO HIGH DRYWELL LEAKAGE FROM THE REACTOR RECIRCULATION SUCTION AND DISCHARGE VALVES, ON 12/12/79 TO ADD A SAFETY RELIEF VALVE POSITIVE INDICATION SYSTEM TO THE PLANT, AND ON 12/21/79 TO CORRECT PROBLEMS FROM THE 12/12/79 OUTAGE.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

BRUNSWICK 2

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	North Carolina/Brunswick 3 miles N of Southport
2.	Electric Reliability Council	Southeastern
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2436
5.	Design Electrical Rating (MWE Net)	821
6.	Date of Initial Criticality	March 20, 1975
7.	Date of Commercial Operation	November 3, 1975
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Cape Fear River

Utility & Contractor Information

10.	Licensee	Carolina Power and Light Company
11.	Corporate Address	411 Fayetteville Street Raleigh, North Carolina 27602
12.	Corporate Contact	J. A. Jones Executive Vice President
13.	Architect/Engineer	United Engineers & Constructors, Inc.
14.	Nuclear Steam Supply System	GE
15.	Constructor	Brown and Root, Inc.

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R.C. Lewis
18.	IE Senior Inspector	P.J. Kellogg
19.	IE Resident Inspector	J. E. Ouzts
20.	Licensing Project Manager	J. Hannon
21.	Docket Number	50-324
22.	License Number/Date of Issue	DPR-62, 12/27/74
23.	Public Document Room	Southport-Brunswick County Library 109 W. Moore Street Southport, North Carolina 28461

INSPECTION STATUS

ENFORCEMENT STATUS

AS REQUIRED BY SECTION 3.7.2.2 OF THE TECHNICAL SPECIFICATIONS, WITH ONE OR MORE OF THE DIESEL GENERATOR SPRAY AND/OR SPRINKLER SYSTEMS INOPERABLE, ESTABLISH A CONTINUOUS FIRE WATCH WITH BACKUP FIRE SUPPRESSION EQUIPMENT WITHIN 1 HOUR. CONTRARY TO THE ABOVE ON OCTOBER 16 THE INSPECTOR FOUND FIRE PROTECTION SPRINKLER AND STANDPIPE SYSTEMS ISOLATED TO DIESEL GENERATOR NO 3. THE ISOLATED SYSTEMS WERE INDICATED BY LOW PRESSURE ANNUNCIATORS FOR THE SYSTEMS. A FOLLOWUP INVESTIGATION REVEALED THAT TWO DELUGE VALVES (DV-15 AND DV-17) HAD BEEN ISOLATED FOR MAINTENANCE BY MANUALLY CLOSING THREE ISOLATION VALVES. THIS ACTION WAS TAKEN WITHOUT NOTIFYING OPERATING PERSONNEL, AND HAVING THE SYSTEMS PLACED IN A LIMITING CONDITION FOR OPERATION (LCO) AND TAKING COMPENSATORY MEASURES REQUIRED BY THE TECHNICAL SPECIFICATIONS. AN LCO WAS ESTABLISHED AFTER THE INSPECTOR FOUND THE CONDITION, NOTIFIED OPERATING PERSONNEL AND THEY HAD INVESTIGATED THE CAUSE OF THE ANNUNICATOR ALARM.
(7939 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

UNIT IS SHUTDOWN FOR REPAIRS.

LAST IE SITE INSPECTION DATE: OCTOBER 27 - NOVEMBER 30, 1979

INSPECTION REPORT NO: 50-324/79-39

SUMMARY

INSPECTION SUMMARY

INSPECTION SEPTEMBER 29 - OCTOBER 26 (79-37): THIS ROUTINE INSPECTION BY THE RESIDENT INSPECTOR INVOLVED 25 MANHOURS OF INSPECTION TIME ON SITE OF PREVIOUSLY IDENTIFIED EVENTS, PLANT OPERATIONS, RADIATION PROTECTION, PHYSICAL PROTECTION, QUALITY ASSURANCE, ORGANIZATION AND ADMINISTRATION AND UNIT 1 MAINTENANCE. OF THE FIVE AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN FOUR AREAS. ONE APPARENT ITEM OF NONCOMPLIANCE WAS IDENTIFIED IN ONE AREA - FAILURE TO RECOGNIZE THAT A LIMITING CONDITION FOR OPERATION WAS ESTABLISHED AND TAKE COMPENSATORY MEASURES REQUIRED BY TECHNICAL SPECIFICATIONS WHEN FIRE PROTECTION DELUGE VALVES WERE MANUALLY ISOLATED, CAUSING ISOLATION OF SECTIONS OF THE SPRINKLER AND STANDPIPE SYSTEMS TO DIESEL GENERATOR NO. 3.

INSPECTION NOVEMBER 15-16 (79-38): THIS SPECIAL, ANNOUNCED INSPECTION INVOLVED 18 INSPECTOR-HOURS ON SITE IN THE AREAS OF SEISMIC ANALYSIS FOR AS-BUILT SAFETY-RELATED PIPING SYSTEMS (IE BULLETIN 79-14). NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION OCTOBER 27 - NOVEMBER 30 (79-39): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 15 INSPECTOR-HOURS ON SITE IN THE AREAS OF PLANT OPERATIONS, PLANT TOURS, RADIATION PROTECTION, PHYSICAL PROTECTION, QUALITY ASSURANCE, ORGANIZATION AND ADMINISTRATION AND FOLLOWUP AND COVERAGE OF UNIT NO. 1 TRIPS. OF THE SIX AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

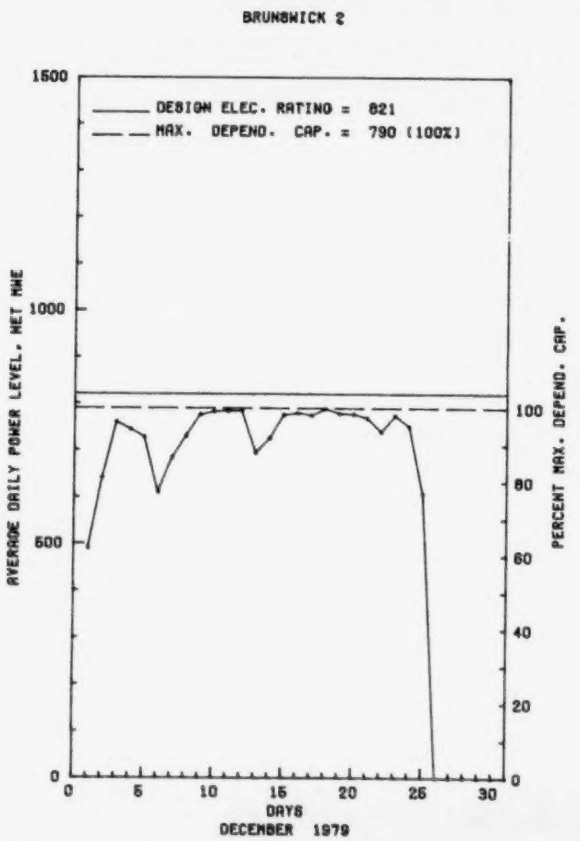
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-095/ 03L-0	11/10/79	12/10/79	30 DAY LER	2A PUMP RELIEF VALVE P029A LIFTED ABOUT 200 PSI BELOW THE SETPOINT DURING A TEST.
79-096/ 03L-0	11/15/79	12/07/79	30 DAY LER	REACTOR LOW PRESSURE CHANNEL CALIBRATION AND FUNCTION TEST SWITCHES 1 AND 2 OUT OF TOLERANCE.
79-099/ 03L-0	11/20/79	12/06/79	30 DAY LER	COOLANT IODINE ACTIVITY EXCEEDED TECHNICAL SPECIFICATIONS LIMIT FOLLOWING BACK TO BACK REACTOR TRIPS.
79-100/ 03L-0	11/20/79	12/13/79	30 DAY LER	DEFECTIVE POSITION INDICATION FOR CONTROL ROD 06-27.

REVIEWED BY OMPA LOEB: John J Rizzo **DATE:** 2-11-80

REVIEWED BY NRR DOR: John N Hannon **DATE:** 2-13-80

BRUNSWICK 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-324
2. Utility Contact: E. Willis (919) 457-9521
3. Licensed Thermal Power (MWt): 2436
4. Nameplate Rating (Gross MWe): 963 x 0.9 = 867
5. Design Electrical Rating (Net MWe): 821
6. Maximum Dependable Capacity (Gross MWe): 815
7. Maximum Dependable Capacity (Net MWe): 790
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	8,760.0	36,481.0
12. Hours Reactor Was Critical	599.8	6,035.7	25,323.9
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	599.7	5,745.0	23,874.0
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,351,546	11,286,627	43,734,462
17. Gross Electrical Energy (MWH)	452,257	3,798,747	14,615,074
18. Net Electrical Energy (MWH)	437,243	3,652,260	14,040,472
19. Unit Service Factor	80.6	65.6	65.4
20. Unit Availability Factor	80.6	65.6	65.4
21. Unit Capacity Factor (MDC Net)	74.4	52.8	48.7
22. Unit Capacity Factor (DER Net)	71.6	50.8	46.9
23. Unit Forced Outage Rate	.0	5.6	13.0

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUELING - 02/15/80 - 1608 HOURS.
25. If Shut Down At End of Report Period, Estimated Date of Startup:

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
024	11/30/79	F	35.2	A	5		HC	HTEXCH	REDUCED POWER FOR A-NORTH, B-NORTH CONDENSER WATERBOX WORK. 2A-NORTH CONDENSER CHECKED FOR LEAKS USING HELIUM METHOD & 8 TUBES FOUND LEAKING. 2B-NORTH CONDENSER CHECKED USING HELIUM METHOD & THE SEAM ACROSS THE TOP OF OUTLET WATERBOX FOUND LEAKING. LEAKING TUBES IN 2A-NORTH PLUGGED & THIS BOX RETURNED TO SATISFACTORY SERVICE. LEAKING SEAM ON 2B-NORTH REPAIRED DURING UNIT #2 SAFETY RELIEF VALVE OUTAGE. OVERTIME WAS WORKED.
025	12/25/79	S	144.3	D	1		SH	VALVEX	THE UNIT HAS SEPARATED FROM THE GRID TO PERFORM A PLANT MODIFICATION TO THE SAFETY RELIEF VALVES. MORE DETAILED INFORMATION WILL BE PROVIDED IN THE REPORT MONTH FOR WHICH THE OUTAGE ENDS.

SUMMARY

THE UNIT BEGAN THE REPORT PERIOD AT REDUCED POWER FOR CONDENSER WATERBOX WORK, AND EXPERIENCED AN OUTAGE ON 12/25/79 TO PERFORM A PLANT MODIFICATION ON THE SAFETY RELIEF VALVES.

LEGEND

[1] Type:
F–Forced
S–Scheduled

[2] Reason:
A–Equipment Failure
B–Maintenance or Test
C–Refueling
D–Regulatory Restriction
E–Operator Training & License Examination
F–Administrative
G–Operational Error
H–Other

[3] Method:
1–Manual
2–Manual Scram
3–Automatic Scram
4–Continuations
5–Load Reduction
9–Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

NRC POWER RESTRICTIONS: NONE

CALVERT CLIFFS 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Maryland/Calvert 40 miles S of Annapolis
2.	Electric Reliability Council	MAAC
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2700
5.	Design Electrical Rating (MWE Net)	845
6.	Date of Initial Criticality	October 7, 1974
7.	Date of Commercial Operation	May 8, 1975
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Chesapeake Bay

Utility & Contractor Information

10.	Licensee	Baltimore Gas and Electric Company
11.	Corporate Address	Gas and Electric Building Baltimore, Maryland 21203
12.	Corporate Contact	A. E. Lundvall, Jr. Vice President-Supply
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Combustion Engineering
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Senior Inspector	R. Keimig
19.	IE Principal Inspector	D. F. Johnson
20.	Licensing Project Manager	E. L. Conner, Jr.
21.	Docket Number	50-317
22.	License Number/Date of Issue	DPR-53, 7/31/74
23.	Public Document Room	Calvert County Library Prince Frederick, Maryland 20678

INSPECTION STATUS

ENFORCEMENT STATUS

THE SLEEVING PLATFORM WAS NOT FABRICATED TO SAFETY RELATED/SEISMIC STANDARDS. 10 CFR 50.59.
(7916 DEFICIENCY)

CONTRARY TO 10 CFR 55, APP A & ACCEPTED REQUALIFICATION PROGRAM, ALL LICENSED OPERATORS DID NOT ATTEND A LECTURE ON DESIGN CHANGES IN 1978. NOR WAS THERE DOCUMENTATION AVAILABLE THAT THEY REVIEWED THESE DESIGN CHANGES.
(7917 DEFICIENCY)

TS 6.11 REQUIRES THAT PROCEDURES FOR RADIATION PROTECTION BE ADHERED TO FOR ALL OPERATIONS INVOLVING PERSONNEL RADIATION EXPOSURE. PROCEDURE RCPR3-310, "PERSONNEL DOSIMETRY QA" REQUIRES THAT AN EVALUATION BE PERFORMED & DOCUMENTED IN CASES WHERE THE DIFFERENCE BETWEEN AN INDIVIDUALS DOSE AS MEASURED BY SELF READING DOSIMETER DISAGREES BY MORE THAN 35% THAN HIS DOSE AS MEASURED BY THERMOLUMINESCENT DOSIMETERS. CONTRARY TO THIS REQUIREMENT, IN 6/79, 12 INDIVIDUALS HAD DOSIMETER DISCREPANCIES OF GREATER THAN 35% AS THE REQUIRED EVALUATION WAS NOT PERFORMED.
(7918 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

Possible oxygen intrusion in September 1979 being evaluated as creating a core crud loading responsible for observed axial power imbalance and core differential pressure increases.

FACILITY ITEMS (PLANS AND PROCEDURES):

None

MANAGERIAL ITEMS:

The following managerial changes were made effective January 1, 1980:

Chairman of the Board and Chief Executive Officer - Bernard C. Trueschler
President and Chief Operating Officer - George V. McGowan
Vice President Electric Interconnection and Operations - John W. Gore, Jr.
Vice President Engineering and Construction - Chris H. Poindexter

PLANT STATUS:

Plant remained at approximately 50% power due to continued evaluation of the axial power imbalance.

LAST IE SITE INSPECTION DATE: 1/21/80

INSPECTION REPORT NO: 79-23

SUMMARY

INSPECTION SUMMARY

Nov 9-11, 13, & 15&16 (79-20): Routine, unannounced inspection by two regional based inspectors of licensee action on previous inspection findings. The inspection involved 2 hours onsite. Noncompliances: None.

Nov 27-30 (79-21): Routine, announced inspection for the purpose of turning over project inspector assignment for the facility. The inspection involved 11 hours onsite by two regional based inspectors. Noncompliances: None.

Dec 6-7 (79-22): Special, unannounced inspection by a regional based inspector (16 hours) to verify the licensee's actions as required by IE Bulletin 79-19 with respect to activities related to the transfer, packaging and shipping of low level radioactive waste materials. Noncompliances: None.

REPORTS RECEIVED FROM LICENSEE

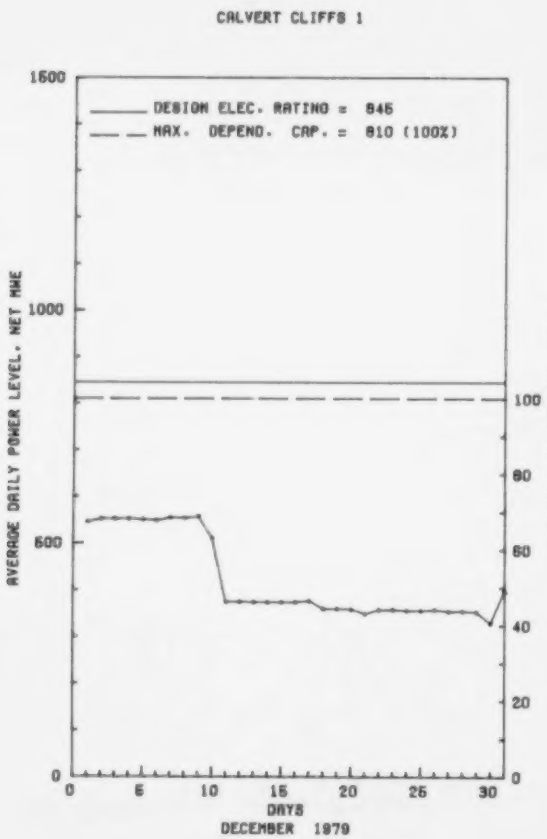
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-052/03L	10/19/79	11/06/79	30 Day LER	Failure of the containment gaseous and particulate radioactivity monitoring system due to loss of the sampling pump
79-053/03L	09/28/79	10/25/79	30 Day LER	As a result of a broken actuator arm #11 shutdown cooling heat exchanger outlet valve was inoperable rendering #11 shutdown cooling heat exchanger inoperable
79-054/03L	09/29/79	10/25/79	30 Day LER	During operation failure of a welded cap on #11 component cooling heat exchanger salt water outlet valve rendered the #11 salt water system inoperable
79-055/03L	009/30/79	11/02/79	30 Day LER	During power operation a module failure caused the channel a power distribution setpoint to decrease. The channel was placed in bypass and subsequently repaired
79-058/03L	10/04/79	11/08/79	30 Day LER	Due to a faulty pneumatic converter #11 salt water outlet valve would not operate properly
79-059/03L	10/26/79	11/13/79	30 Day LER	During routine surveillance two fire hoses were found damaged. Both hoses were replaced
79-060/01L	10/20/79	11/15/79	30 Day LER	During a load test on #12 diesel generator high exhaust temperatures were experienced which resulted in the inability to achieve full load capability of 2500 KW
79-061/01T	10/24/79	11/07/79	Special	As a result of followup on IEB 79-14 it was discovered that the air start systems for the diesel generators did not meet seismic requirements
79-063/04T	10/22/79	11/05/79	14 Day LER	Oyster samples collected on August 27, 1979 showed the presence of Ag-110M at 266 pCi/KG
79-064/03L	11/01/79	11/13/79	30 Day LER	Fire barriers between the service building and intake structure were temporarily removed to facilitate cable installation
79-065/03L	11/13/79	12/04/79	30 Day LER	Emergency diesel generator removed from service to repair Woodward Governor
79-066/03L	11/20/79	12/10/79	30 Day LER	Containment cooler fan inoperable (hand switch failure)
79-068/03L	11/27/79	12/13/79	30 Day LER	No. 12 emergency diesel inoperable to repair service water vent valve
79-069/03L	11/27/79	12/13/79	30 Day LER	No. 11 emergency diesel generator inoperable leaking fuel oil line
79-071/03L	11/11/79	12/06/79	30 Day LER	Plant trip caused by 500 KV breaker malfunction
79-072/03L	11/16/79	12/06/79	30 Day LER	500 KV transmission line deenergized
79-075/04T	11/29/79	12/13/79	14 Day LER	Waste neutralizing tank overflowed resulting in unmonitored discharge

REVIEWED BY OMPA LOEB: MR Bula DATE: 02/11/80

REVIEWED BY NRR DOR: E L Conner DATE:

CALVERT CLIFFS 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-317
2. Utility Contact: S. D. Merson (301) 234-5240
3. Licensed Thermal Power (MWt): 2700
4. Nameplate Rating (Gross MWe): 1020 x 0.9 = 918
5. Design Electrical Rating (Net MWe): 845
6. Maximum Dependable Capacity (Gross MWe): 845
7. Maximum Dependable Capacity (Net MWe): 810
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): VARIOUS
10. Reasons for Restrictions, If Any: CRUD BUILDINGUP IN CORE HAS ADVERSELY AFFECTED THE POWER DISTRIBUTION

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	8,760.0	40,765.0
12. Hours Reactor Was Critical	744.0	6,351.0	32,439.6
13. Reactor Reserve Shutdown Hours	.0	166.9	1,060.5
14. Hours Generator On-Line	744.0	6,157.0	31,702.0
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,126,318	14,066,784	75,336,988
17. Gross Electrical Energy (MWH)	338,790	4,426,195	24,812,530
18. Net Electrical Energy (MWH)	313,511	4,194,216	23,649,206
19. Unit Service Factor	100.0	70.3	77.8
20. Unit Availability Factor	100.0	70.3	77.8
21. Unit Capacity Factor (MDC Net)	52.0	59.1	71.6
22. Unit Capacity Factor (DER Net)	49.9	56.7	68.7
23. Unit Forced Outage Rate	.0	14.1	9.0

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): TMI MODS - 01/25/80 - 6 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
79-15	11/08/79	F	744.0	H	5		RC	ZZZZZZ	FORCED REDUCTIONS TO VARIOUS LOADS DUE TO UNEQUAL POWER DISTRIBUTION IN CORE.

SUMMARY

THE UNIT OPERATED AT VARIOUS FORCED LOAD REDUCTIONS THROUGHOUT THE REPORT PERIOD, DUE TO UNEQUAL POWER DISTRIBUTION OF THE CORE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

CALVERT CLIFFS 2

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Maryland/Calvert 40 miles S of Annapolis
2.	Electric Reliability Council	MAAC
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2700
5.	Design Electrical Rating (MWE Net)	845
6.	Date of Initial Criticality	November 30, 1976
7.	Date of Commercial Operation	April 1, 1977
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Chesapeake Bay

Utility & Contractor Information

10.	Licensee	Baltimore Gas and Electric Company
11.	Corporate Address	Gas and Electric Building Baltimore, Maryland 21203
12.	Corporate Contact	A. E. Lundvall, Jr. Vice President-Supply
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Combustion Engineering
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	IE Senior Inspector	R. Keimig
19.	IE Principal Inspector	D. F. Johnson
20.	Licensing Project Manager	E. L. Conner, Jr.
21.	Docket Number	50-318
22.	License Number/Date of Issue	DPR-69, 8/13/76
23.	Public Document Room	Calvert County Library Prince Frederick, Maryland 20678

INSPECTION STATUS

ENFORCEMENT STATUS

10 CFR 50.59 STATES "THE HOLDER OF A LICENSE AUTHORIZING OPERATION OF A UTILIZATION FACILITY MAY MAKE CHANGES IN THE FACILITY AS DESCRIBED IN THE SAFETY ANALYSES REPORT WITHOUT PRIOR COMMISSION APPROVAL, UNLESS THE PROPOSED CHANGE INVOLVES A CHANGE IN TS OR AN UNREVIEWED SAFETY QUESTION. A WRITTEN SAFETY EVALUATION PROVIDES THE BASIS FOR THE DETERMINATION THAT THE CHANGE DOES NOT INVOLVE AN UNREVIEWED SAFETY QUESTION." THE SAFETY EVALUATION FOR MODIFICATION FCR-78-26, FABRICATION & INSTALLATION OF A SLEEVING PLATFORM, STATES THAT AN UNREVIEWED SAFETY QUESTION IS NOT INVOLVED SINCE THE PLATFORM WAS FABRICATED & INSTALLED TO SAFETY RELATED/SEISMIC CRITERIA. CONTRARY TO THE ABOVE, THE SLEEVING PLATFORM WAS NOT FABRICATED TO SAFETY RELATED/SEISMIC STANDARDS.
(7913 DEFICIENCY)

TS 6.11 REQUIRES THAT PROCEDURES FOR RADIATION PROTECTION BE ADHERED TO FOR ALL OPERATIONS INVOLVING PERSONNEL RADIATION EXPOSURE. PROCEDURE RCP3-310, "PERSONNEL DOSIMETRY QA" REQUIRES THAT AN EVALUATION BE PERFORMED & DOCUMENTED IN CASES WHERE THE DIFFERENCE BETWEEN AN INDIVIDUAL DOSE AS MEASURED BY SELF READING DOSIMETER DISAGREES BY MORE THAN 35% THAN HIS DOSE AS MEASURED BY THERMOLUMINESCENT DOSIMETERS. CONTRARY TO THIS REQUIREMENT, IN 6/79, 12 INDIVIDUALS HAD DOSIMETER DISCREPANCIES OF GREATER THAN 35%. THE REQUIRED EVALUATION WAS NOT PERFORMED.
(7915 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

None

FACILITY ITEMS (PLANS AND PROCEDURES):

None.

MANAGERIAL ITEMS:

- The following managerial changes were made effective January 1, 1980:

  Chairman of the Board and Chief Executive Officer - Bernard C. Trueschler
  President and Chief Operating Officer - George V. McGowan
  Vice President Electric Interconnection and Operations - John W. Gore, Jr.
  Vice President Engineering and Construction - Chris H. Poindexter

PLANT STATUS:

- Plant was shutdown on January 11-21 to complete modifications required by TMI-2 Short Term Lessons Learned.

- LAST IE SITE INSPECTION DATE: 1/21/80

- INSPECTION REPORT NO: 79-22

SUMMARY

INSPECTION SUMMARY

- Nov 8-9 (79-18): Routine, unannounced inspection of in-service inspection activities. The inspection involved 12 inspector-hours onsite by one NRC inspector. Noncompliances: None.

- Nov. 9-11, 13, & 15-16 (79-19): Routine, unannounced inspection by two regional based inspectors (48 hours) of the Containment Integrated Leak Rate Test - CILRT and licensee action on previous inspection findings. Noncompliances: None.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-037/03L	10/05/79	11/05/79	30 Day LER	During normal operation ESFAS sensor cabinet 2F was deenergized temporarily to replace a failed voltage meter
79-038/03L	10/22/79	11/20/79	30 Day LER	Shutdown cooling was lost for approximately 3 minutes due to a personnel error in removing the wrong pressure transmitter from service
79-039/01T	10/24/79	11/07/79	Special	As a result of followup on IEB 79-14 it was discovered that the air start systems for the diesel generator did not meet seismic requirements
79-040/03L	10/12/79	11/06/79	30 Day LER	#21 diesel generator was taken out of service temporarily to repair a faulty voltage regulator
79-041/03L	10/27/79	11/15/79	30 Day LER	#22 high pressure safety injection pump placed out of service due to mechanical seal failure

- Nov 27-30 (79-20): Routine, announced inspection for the purpose of turning over project inspector assignment for the facility. The inspection involved 11 hours onsite by two regional based inspectors. Noncompliances: None.

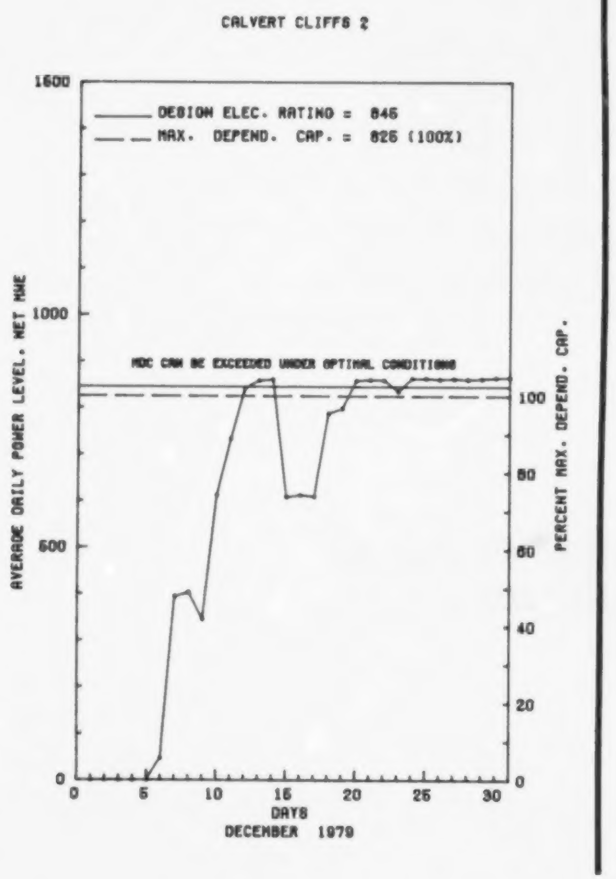
- Dec 6-7 (79-21): Special, unannounced inspection by a regional based inspector (16 hours) to verify the licensee's actions as required by IE Bulletin 79-19 with respect to activities related to the transfer, packaging and shipping of low level radioactive waste materials. Noncompliances: None.

REVIEWED BY OMPA LOEB: MR Beebe DATE: 02/11/80

REVIEWED BY NRR DOR: E L Brunner DATE: 2/13/80

CALVERT CLIFFS 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-318
2. Utility Contact: S. D. Merson (301) 234-5240
3. Licensed Thermal Power (MWt): 2700
4. Nameplate Rating (Gross MWe): 1012 x 0.9 = 911
5. Design Electrical Rating (Net MWe): 845
6. Maximum Dependable Capacity (Gross MWe): 860
7. Maximum Dependable Capacity (Net MWe): 825
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	8,760.0	24,120.0
12. Hours Reactor Was Critical	646.8	6,952.4	19,944.9
13. Reactor Reserve Shutdown Hours	.0	193.7	413.1
14. Hours Generator On-Line	606.6	6,796.7	15,608.5
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,411,500	17,360,215	48,244,231
17. Gross Electrical Energy (MWH)	473,323	5,751,151	16,005,223
18. Net Electrical Energy (MWH)	448,389	5,488,991	15,257,020
19. Unit Service Factor	81.5	77.6	81.3
20. Unit Availability Factor	81.5	77.6	81.3
21. Unit Capacity Factor (MDC Net)	73.1	76.0	76.7
22. Unit Capacity Factor (DER Net)	71.3	74.2	74.9
23. Unit Forced Outage Rate	18.4	8.8	6.9

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): TMI MODS - 1/11/80 THRU 1/25/80
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
79-18	11/26/79	F	136.4	B	4		CB	PUMPXX	REPLACE SEALS ON NOS. 21B AND 22B REACTOR COOLANT PUMPS.
79-19	12/10/79	S	1.0	B	2		XX	ZZZZZZ	OVERSPEED TRIP TEST ON THE TURBINE.
79-20	12/15/79	S	61.0	B	5		XX	ZZZZZZ	TO CONDUCT POWER COEFFICIENT MEASUREMENTS.
79-21	12/17/79	F	51.0	B	5		XX	ZZZZZZ	TO CONDUCT POWER COEFFICIENT MEASUREMENTS.

SUMMARY

OPERATION WAS ROUTINE FOLLOWING RETURN TO SERVICE ON 12/06/79 FROM LAST PERIOD'S OUTAGE FOR SEAL REPLACEMENT ON REACTOR COOLANT PUMPS. LOAD REDUCTIONS OCCURRED ON 12/10/79 FOR TURBINE OVERSPEED COEFFICIENT MEASUREMENT.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

COOK 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Michigan/Berrien 11 miles S of Benton Harbor
2.	Electric Reliability Council	ECAR
3.	Type of Reactor	PWR
4.	Power Level (MWT)	3250
5.	Design Electrical Rating (MWE Net)	1054
6.	Date of Initial Criticality	January 18, 1975
7.	Date of Commercial Operation	August 27, 1975
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10.	Licensee	AEP Corporation
11.	Corporate Address	Indiana & Michigan Power Co. 2 Broadway New York, New York 10004
12.	Corporate Contact	John Dolan Senior Executive V. P. Eng.
13.	Architect/Engineer	AEP
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Indiana & Michigan Power Co.

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Senior Inspector	D.C. Boyd
19.	IE Resident Inspector	R. Masse (616) 465-6775
20.	Licensing Project Manager	D. Wigginton
21.	Docket Number	50-315
22.	License Number/Date of Issue	DPR-58 issued 10/25/74
• 23.	Public Document Room	Maude Preston Palenske Memorial Library 500 Market Street St. Joseph, Michigan 49085

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

Plant shutdown on December 24, 1979, when design basis earthquake analysis data indicated that the containment air recirculation system ducting (12 inch pipe) would be stressed beyound yield. Start-up is expected on January 2, 1980.

LAST IE SITE INSPECTION DATE: September 1 through October 5, 1979

INSPECTION REPORT NO: 79-23

SUMMARY

INSPECTION SUMMARY

No inspections conducted.

REPORTS RECEIVED FROM LICENSEE

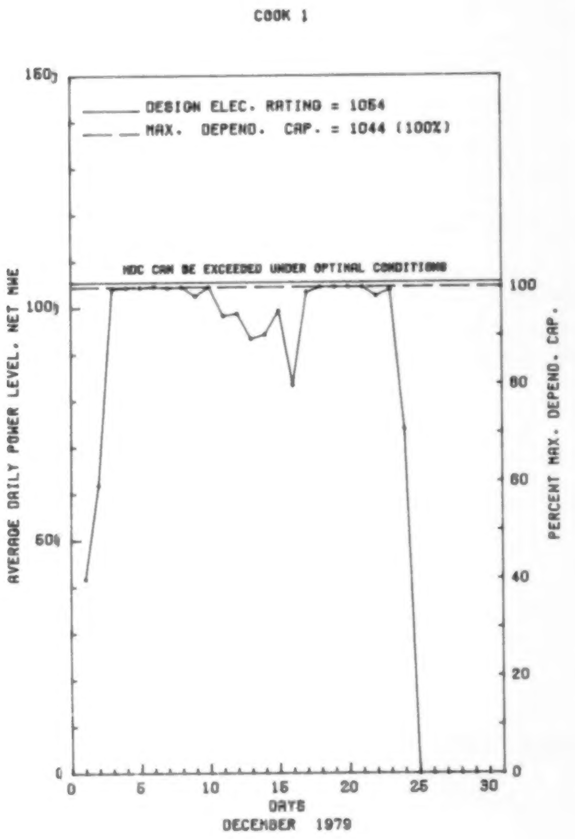
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-57/ 03L-0	10/29/79	11/28/79	30 Day LER	While shifting to RHR during a normal cooldown, motor operated containment isolation valve, ICM-111 (RHR Disch. to cold legs 2 & 3), failed to open completely and then would not close.
79-58/ 01T-0	11/15/79	11/29/79	14 Day LER	During normal operations while the turbine drive AFP was out of service for governor repairs, the flow path from the Unit 2 motor driven AFP to No. 1 and No. 4 steam generators on Unit 1 was made inoperable when the power supply to motor operated valves FMO-212 and FMO-242 was removed.
79-59/ 03L-0	11/08/79	12/07/79	30 Day LER	During the performance of a surveillance test procedure, ECR-31 (containment isolation valve for the containment air particulate detector) would not close.
79-60/ 03L-0	11/16/79	12/07/79	30 Day LER	During a routine ice condenser tour on two consecutive days, one intermediate deck door was found frozen shut.
79-61/ 03L-0	11/11/79	12/11/79	30 Day LER	During the startup of Unit 1, No. 4 steam flow channel I was found to be indicating approx. 21% lower than the redundant channel.
79-62/ 03L-0	11/11/79	12/11/79	30 Day LER	An inoperable steam flow channel (Channel I) was taken out of the trip condition for a period of 25 min.

REVIEWED BY OMPA LOEB: T. C. Cintula DATE: 2·11·80

REVIEWED BY NRR DOR: DATE: 2.13.80

COOK 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-315
2. Utility Contact: W. T. Gillett (616) 465-5901
3. Licensed Thermal Power (MWt): 3250
4. Nameplate Rating (Gross MWe): 1280 x 0.9 = 1152
5. Design Electrical Rating (Net MWe): 1054
6. Maximum Dependable Capacity (Gross MWe): 1080
7. Maximum Dependable Capacity (Net MWe): 1044
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	8,760.0	42,624.0
12. Hours Reactor Was Critical	557.9	5,750.0	32,961.7
13. Reactor Reserve Shutdown Hours	.0	.0	463.0
14. Hours Generatur On-Line	555.7	5,669.9	32,083.2
15. Unit Reserve Shutdown Hours	.0	.0	321.0
16. Gross Thermal Energy (MWH)	1,723,754	17,728,514	89,066,305
17. Gross Electrical Energy (MWH)	573,290	5,867,680	29,157,340
18. Net Electrical Energy (MWH)	553,020	5,660,137	27,997,314
19. Unit Service Factor	74.7	64.7	75.1
20. Unit Availability Factor	74.7	64.7	75.1
21. Unit Capacity Factor (MDC Net)	71.2	61.9	67.0
22. Unit Capacity Factor (DER Net)	70.5	61.3	62.0
23. Unit Forced Outage Rate	25.3	9.9	6.9

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 1/2/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
153	12/01/79	F	13.7	G	3		RB	INSTRU	WHILE WORKING ON ROD CONTROL SYSTEM TO CLEAR A "ROD CONTROL URGENT FAILURE" ALARM A WRONG CARD WAS PULLED, DROPPING THE RODS IN THAT GROUP WHICH CAUSED A "NEGATIVE RATE" REACTOR TRIP. UNIT WAS RETURNED TO SERVICE THE SAME DAY AND REACHED 100% POWER ON 12/02/79.
154	12/16/79	S	.0	B	4		ZZ	ZZZZZZ	REACTOR POWER REDUCED TO 70% TO PERFORM N.I.S. INCORE/EXCORE CROSS CALIBRATION. POWER RETURNED TO 100% 12/17/79.
155	12/24/79	F	174.6	H	1	79-065	SE	SUPORT	UNIT SHUTDOWN AND BROUGHT TO COLD SHUTDOWN (MODE 5) DUE TO A SIGNIFICANT NON-CONFORMANCE IDENTIFIED DURING INSPECTION/EVALUATION PROGRAM PERFORMED IN ACCORDANCE WITH IE BULLETIN NO. 79-14.

SUMMARY

OPERATION WAS AT NEAR FULL POWER UNTIL 12/24/79 WHEN THE UNIT WAS SHUTDOWN DUE TO DESIGN DEFICIENCIES DISCOVERED IN THE CONTAINMENT HYDROGEN SKIMMER SYSTEM. LOAD REDUCTIONS OCCURRED DUE TO A NEGATIVE RATE REACTOR TRIP AND INCORE/EXCORE CROSS CALIBRATION.

LEGEND

[1] Type:
F–Forced
S–Scheduled

[2] Reason:
A–Equipment Failure
B–Maintenance or Test
C–Refueling
D–Regulatory Restriction
E–Operator Training & License Examination
F–Administrative
G–Operational Error
H–Other

[3] Method:
1–Manual
2–Manual Scram
3–Automatic Scram
4–Continuations
5–Load Reduction
9–Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

COOK 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1. Location (state/county) (nearest population center)	Michigan/Berrien 11 miles S of Benton Harbor
2. Electric Reliability Council	ECAR
3. Type of Reactor	PWR
4. Power Level (MWT)	3391
5. Design Electrical Rating (MWE Net)	1100
6. Date of Initial Criticality	March 10, 1978
7. Date of Commercial Operation	July 1, 1978
8. Condenser Cooling Method	Once Through
9. Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10. Licensee	AEP Corporation Indiana & Michigan Power Co. Indiana & Michigan Electric Co.
11. Corporate Address	2 Broadway New York, New York 10004
12. Corporate Contact	John Dolan, VP/Chief Engineer (212) 422 - 4800
13. Architect/Engineer	American Elec. Power Service Corp.
14. Nuclear Steam Supply System	Westinghouse
15. Constructor	Indiana & Michigan Power Co.

Regulatory Information

16. IE Region responsible	III
17. IE Branch Chief	R. Heishman
18. IE Senior Inspector	D.C. Boyd
19. IE Resident Inspector	R. Masse (616) 465-6775
20. Licensing Project Manager	D. Wigginton
21. Docket Number	50-316
22. License Number/Date of Issue	DPR-74, 12/23/77
23. Public Document Room	Maude Preston Palenske Memorial Library 500 Market Street St. Joseph, Michigan 49085

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

Reactor shutdown on October 19, 1979 to begin a 65-day refueling and maintenance outage.

LAST IE SITE INSPECTION DATE: September 1 through October 5, 1979

INSPECTION REPORT NO: 79-20

SUMMARY

INSPECTION SUMMARY

No inspections conducted.

REPORTS RECEIVED FROM LICENSEE

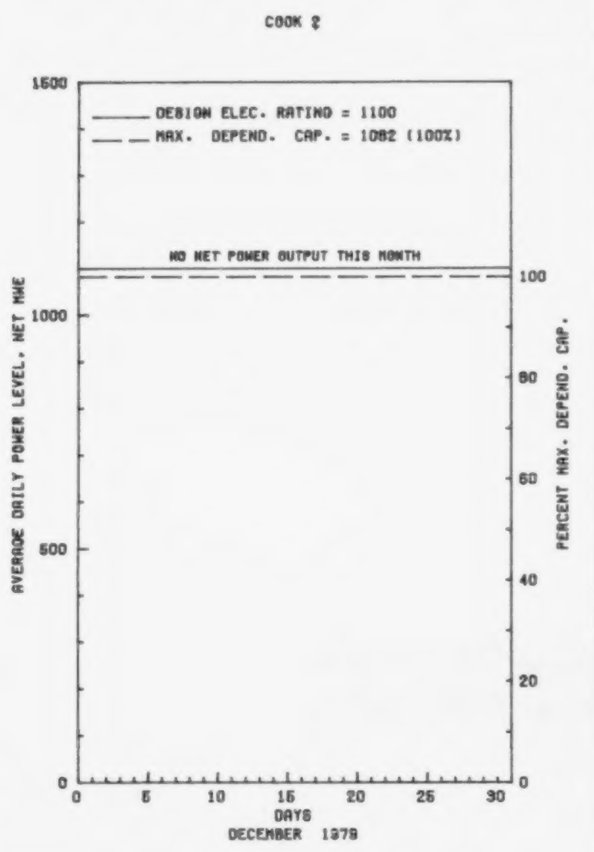
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPROT	SUBJECT
79-38/01T-0	11/14/79	11/28/79	14 Day LER	During refueling operations the boron injection flow path required by TS 3.1.2.1 was inadvertently made inoperable to perform surveillance testing.
79-39/03L-0	10/31/79	11/28/79	30 Day LER	The containment air particulate detector (R-11) tripped on low flow while the plant was in Mode 6.
79-40/03L-0	11/01/79	11/29/79	30 Day LER	During refueling operations the reactor vessel head was unbolted with one source range neutron flux monitor inoperable.
79-41/03L-0	11/01/79	11/30/79	30 Day LER	During surveillance testing, hydrogen recombiner No. 1 failed to increase temp. greater than or equal to 1200 degrees F within 5 hours as required by TS 4.6.4.2B3.
79-42/03L-0	11/01/79	12/03/79	30 Day LER	While in mode 6, with one diesel generator (DG) out of service contractor personnel mistakenly began to determinate the operable DG's starting bus feed cable, blowing a fuse in the 1KV inverter.
79-43/03L-0	11/02/79	12/03/79	30 Day LER	During surveillance inspection per TS 4.6.5.9, the ice condenser divider barrier seal was found to have cracks in the rubber.
79-44/03L-0	10/25/79	12/07/79	30 Day LER	While transferring fuel from the new fuel vault to the spent fuel pit, the surveillance requirement for determining potential impact energy of the crane's load (TS 4.9.7.2) was not completed.

REVIEWED BY OMPA LOEB: T.C. Cintula DATE: 2·11·80

REVIEWED BY NRR DOR: D Wigginton DATE: 2·13·80

COOK 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-316
2. Utility Contact: W. T. Gillett (616) 465-5901
3. Licensed Thermal Power (MWt): 3391
4. Nameplate Rating (Gross MWe): 1333 x 0.85 = 1133
5. Design Electrical Rating (Net MWe): 1100
6. Maximum Dependable Capacity (Gross MWe): 1118
7. Maximum Dependable Capacity (Net MWe): 1082
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	8,760.0	17,520.0
12. Hours Reactor Was Critical	.0	5,849.2	11,879.9
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	.0	5,774.3	10,581.9
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	19,119,273	32,224,380
17. Gross Electrical Energy (MWH)	0	6,168,880	10,154,410
18. Net Electrical Energy (MWH)	0	5,953,413	9,767,412
19. Unit Service Factor	.0	65.9	60.7
20. Unit Availability Factor	.0	65.9	60.7
21. Unit Capacity Factor (MDC Net)	.0	62.8	63.7
22. Unit Capacity Factor (DER Net)	.0	61.8	62.6
23. Unit Forced Outage Rate	.0	17.2	13.8

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 01/12/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
66	10/19/79	S	744.0	C	4		ZZ	ZZZZZZ	THE UNIT WAS REMOVED FOR CYCLE I-II REFUELING OUTAGE ON 10/19/79. THE OUTAGE WAS EXTENDED ON 12/23/79 TO CORRECT DISCREPANCIES FOUND AS A RESULT OF SEISMIC ANALYSIS FOR AS-BUILT SAFETY-RELATED PIPING SYSTEMS PERFORMED IN ACCORDANCE WITH I.E. BULLETIN NO. 79-14.

SUMMARY

THE UNIT REMAINED SHUTDOWN THROUGHOUT THE REPORT PERIOD FOR ITS FIRST REFUELING OUTAGE, AND TO MAKE MODIFICATIONS IN SAFETY-RELATED PIPING SYSTEMS PER NRC BULLETIN NO. 79-14.

LEGEND

[1] Type:
F–Forced
S–Scheduled

[2] Reason:
A–Equipment Failure
B–Maintenance or Test
C–Refueling
D–Regulatory Restriction
E–Operator Training & License Examination
F–Administrative
G–Operational Error
H–Other

[3] Method:
1–Manual
2–Manual Scram
3–Automatic Scram
4–Continuations
5–Load Reduction
9–Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

COOPER STATION

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Nebraska/Nemaha 23 miles S of Nebraska City
2.	Electric Reliability Council	MARCA
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2381
5.	Design Electrical Rating (MWE Net)	778
6.	Date of Initial Criticality	February 21, 1974
7.	Date of Commercial Operation	July 1, 1974 (50% rating)
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Missouri River

Utility & Contractor Information

10.	Licensee	Nebraska Public Power District
11.	Corporate Address	P. O. Box 499 Columbus, Nebraska 68601
12.	Corporate Contact	J. M. Pilant, Director Licensing and Quality Assurance
13.	Architect/Engineer	Burns and Roe, Inc.
14.	Nuclear Steam Supply System	GE
15.	Constructor	Burns and Roe, Inc.

Regulatory Information

16.	IE Region responsible	IV
17.	IE Branch Chief	G. L. Madsen
18.	IE Senior Inspector	T. F. Westerman
19.	IE Principal Inspector	G. L. Constable
20.	Licensing Project Manager	V. L. Rooney
21.	Docket Number	50-298
22.	License Number/Date of Issue	DPR-46, 1/18/74
23.	Public Document Room	Auburn Public Library 118 15th Street Auburn, Nebraska 68305

INSPECTION STATUS

ENFORCEMENT STATUS

TECHNICAL SPECIFICATION 6.8.1.A SPECIFIES THAT THE LICENSEE SHALL ESTABLISH, IMPLEMENT, AND MAINTAIN WRITTEN PROCEDURES COVERING THE ACTIVITIES IN APPENDIX A OF REGULATORY GUIDE 1.33. STATION ADMINISTRATIVE INSTRUCTION AI-500 SECTION 2.7 STATES "WHEN AN OPERATOR HAS PERFORMED AN OPERATION OR VERIFIED THE STATUS OF THE ITEM AS REQUIRED BY THE SPECIFIC PROCEDURAL STEP, HE INITIALS THE SPACE PROVIDED FOR THAT STEP IN THE PROCEDURE AS ON A CHECK-OFF LIST". CONTRARY TO THE ABOVE STEP II-A OF RADIATION RELEASE PERMIT RBVRP-79-55 WAS SIGNED ON OCTOBER 19, 1979 INDICATING THAT RADIATION MONITOR RM-A1(G) SETPOINT AND ALERT VALUES HAD BEEN VERIFIED AND THAT THE MONITOR WAS OPERATIONAL. ACTUALLY THE SETPOINT AND ALERT VALUES WERE TEN TIMES THE VALUES LISTED ON THE RELEASE PERMIT.
(7945 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

FACILITY ITEMS (PLANS AND PROCEDURES):

MANAGERIAL ITEMS:

PLANT STATUS:

Unit operating in load following mode up to full power.

LAST IE SITE INSPECTION DATE: November 14 - 16, 1979 *

INSPECTION REPORT NO: 50-298/79-16 *

SUMMARY

INSPECTION SUMMARY

Inspection conducted October 23-26, November 5-7, 14-16, 1979 (79-16): Unannounced inspection of licensee's document control, Station Operations Review Committee, records program, review of plant operations, spent fuel storage racks, and follow-up on LER's and IE Bulletins. The inspection involved fifty-six (56) hours on site by one NRC inspector. The licensee was found to be in compliance with NRC requirements within the 6 areas examined during the inspection.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

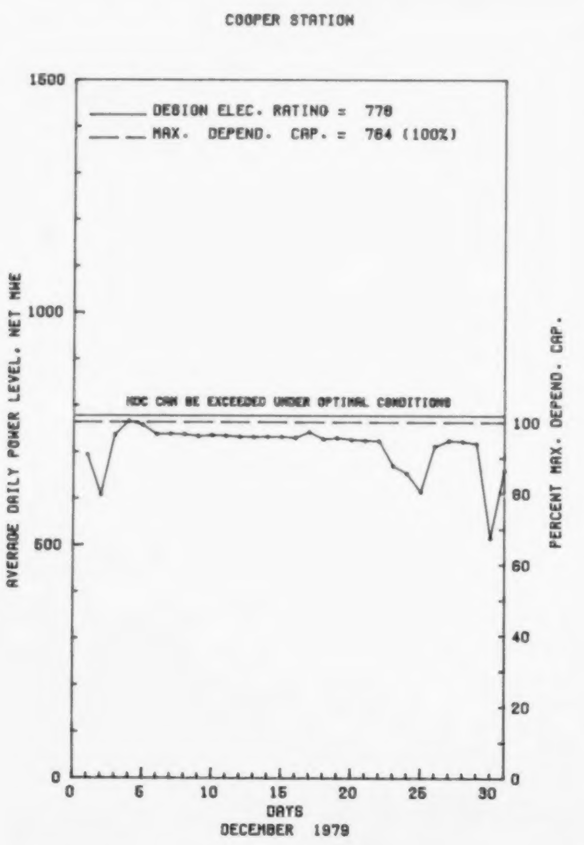
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
79-53 03L-0	11/10/79	12/6/79	Failure to test No. 1 D.G. O.O.C.
79-54 03L-0	11/7/79	12/6/79	Broken oil hose to am shaft chain tightener
79-55 03L-0	11/9/79	12/7/79	Core spray motor oprated valve failed
79-36 03L-0	11/18/79	12/10/79	D.G. No. 2 cylinder sleeve damage
79-37 03L-0	11/13/79	12/17/79	Silencer bypass solnoid failure

REVIEWED BY OMPA LOEB: DATE: 2-11-80

REVIEWED BY NRR DOR: DATE: 2/13/80

COOPER STATION

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-298
2. Utility Contact: P. J. Borer (402) 825-3811
3. Licensed Thermal Power (MWt): 2381
4. Nameplate Rating (Gross MWe): 983 x 0.85 = 836
5. Design Electrical Rating (Net MWe): 778
6. Maximum Dependable Capacity (Gross MWe): 787
7. Maximum Dependable Capacity (Net MWe): 764
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	8,760.0	48,241.0
12. Hours Reactor Was Critical	744.0	7,782.4	41,237.3
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	744.0	7,672.4	40,475.6
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,640,784	15,758,568	76,182,726
17. Gross Electrical Energy (MWH)	547,414	5,180,962	24,538,848
18. Net Electrical Energy (MWH)	528,357	4,994,938	23,658,277
19. Unit Service Factor	100.0	87.6	83.9
20. Unit Availability Factor	100.0	87.6	83.9
21. Unit Capacity Factor (MDC Net)	93.0	74.6	64.2
22. Unit Capacity Factor (DER Net)	91.3	73.3	63.0
23. Unit Forced Outage Rate	.0	3.8	4.6

24. Shutdowns Scheduled Over Next 6 Months (Type, Date and Duration of Each):
REFUELING & MAINT. MARCH 15, 1980, 8 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
79-15	12/30/79	S	0	H	4				POWER WAS REDUCED TO ALLOW TESTING OF THE TURBINE CONTROL SYSTEM AND ADJUST THE CONTROL ROD PATTERN.

SUMMARY

OPERATIONS WAS AT NEAR FULL POWER DURING THE REPORT PERIOD, WITH A LOAD REDUCTION ON 12/30/79 FOR TURBINE CONTROL SYSTEM TESTING AND CONTROL ROD ADJUSTMENT.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

CRYSTAL RIVER 3

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Florida/Citrus 7 miles NW of Crystal River
2.	Electric Reliability Council	Southeastern
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2452
5.	Design Electrical Rating (MWE Net)	825
6.	Date of Initial Criticality	January 14, 1977
7.	Date of Commercial Operation	March 13, 1977
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Gulf of Mexico

Utility & Contractor Information

10.	Licensee	Florida Power Corporation
11.	Corporate Address	3201 34th Street, South St. Petersburg, Florida 33733
12.	Corporate Contact	J. Hancock
13.	Architect/Engineer	Gilbert Associates, Inc.
14.	Nuclear Steam Supply System	B&W
15.	Constructor	J. A. Jones Construction Co.

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. Lewis
18.	IE Senior Inspector	E. H. Verdery
19.	IE Principal Inspector	D. R. Quick
20.	Licensing Project Manager	M. B. Fairtile
21.	Docket Number	50-302
22.	License Number/Date of Issue	DPR-72, 12/3/76
23.	Public Document Room	Crystal River Public Library 668 N.W. First Crystal River, Florida 32639

INSPECTION STATUS

ENFORCEMENT STATUS

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

NORMAL OPERATION AT 100% POWER - NO APPARENT MAJOR PROBLEMS.

LAST IE SITE INSPECTION DATE: DECEMBER 17-19, 1979

INSPECTION REPORT NO: 50-302/79-54

SUMMARY

INSPECTION SUMMARY

INSPECTION OCTOBER 22-26 (79-45): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 30 INSPECTOR-HOURS ON SITE IN THE AREAS OF AUDITS OF PLANT OPERATIONS, OPERATIONS RECORDS REVIEW, SEISMIC RESTRAINT MODIFICATION AND PLANT TOUR. OF THE FOUR AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN THREE AREAS; ONE APPARENT ITEM OF NONCOMPLIANCE WAS FOUND IN ONE AREA (INFRACTION - FAILURE TO PROPERLY SET THE ALARMS OF RADIATION MONITORS IN ACCORDANCE WITH THE RELEASE PERMIT PRIOR TO INITIATING A REACTOR BUILDING VENTILATION PURGE. THE PURGE WAS SECURED WHEN THE RADIATION LEVELS REACHED THE HIGHER SET POINTS).

INSPECTION NOVEMBER 5-9 (79-47): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 27 INSPECTOR-HOURS ON SITE IN THE AREAS OF PLANT OPERATIONS AND FOLLOWUP ON PREVIOUSLY IDENTIFIED ITEMS. OF THE TWO AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION NOVEMBER 13-16 (79-48): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 32 INSPECTOR-HOURS ON SITE IN THE AREAS OF FOLLOWUP ON PREVIOUS INSPECTION FINDINGS, REVIEW OF PLANT OPERATIONS, REVIEW OF ELECTRICAL POWER DISTRIBUTION SYSTEM FOR THE ICS AND NNI SYSTEMS, AND SURVEILLANCE MONITORING. AN ADDITIONAL TWENTY HOURS WERE SPENT IN THE REGIONAL OFFICE REVIEWING THE PLANT'S INTEGRATED CONTROL SYSTEM AND NONNUCLEAR INSTRUMENTATION SYSTEM AND THEIR RESPONSE TO A LOSS OF AC POWER. OF THE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION NOVEMBER 19-21 (79-51): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 18 INSPECTOR-HOURS ON SITE IN THE AREAS OF PLANT OPERATIONS AND REVIEW OF LICENSEE EVENT REPORTS. OF THE TWO AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

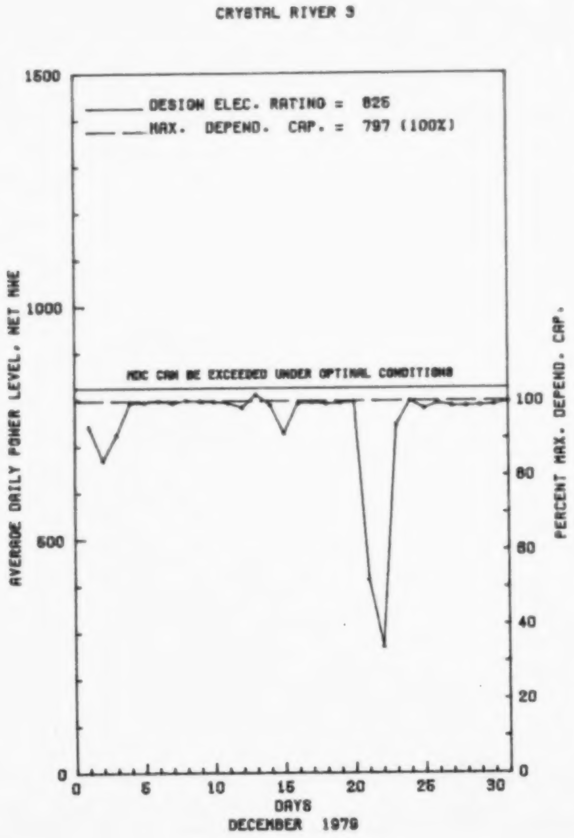
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-105/ 03L-0	11/01/79	12/06/79	30 DAY LER	CONTAINMENT ISOLATION VALVE ON CHEMICAL ADDITION LINE (CAV-2) FAILED TO CLOSE.
79-106/ 03L-0	11/08/79	12/04/79	30 DAY LER	HEAT TRACE RECORDER HTRT-5 FAILED. THIS RECORDER SERVES CONCENTRATED BORIC ACID FLOWPATH.

REVIEWED BY OMPA LOEB: P E Bohe **DATE:** 2/11/80

REVIEWED BY NRR DOR: M. B. Fairtile **DATE:** 2/14/80

CRYSTAL RIVER 3

AVERAGE POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-302
2. Utility Contact: W. A. Stephenson (904) 795-6486
3. Licensed Thermal Power (MWt): 2452
4. Nameplate Rating (Gross MWe): 989 x 0.9 = 890
5. Design Electrical Rating (Net MWe): 825
6. Maximum Dependable Capacity (Gross MWe): 835
7. Maximum Dependable Capacity (Net MWe): 797
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	8,760.0	24,576.0
12. Hours Reactor Was Critical	731.2	5,293.1	15,150.2
13. Reactor Reserve Shutdown Hours	.0	733.5	1,116.9
14. Hours Generator On-Line	720.6	5,155.6	14,700.3
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,726,073	11,719,743	32,316,911
17. Gross Electrical Energy (MWH)	588,258	3,973,897	10,971,743
18. Net Electrical Energy (MWH)	560,272	3,761,775	10,391,640
19. Unit Service Factor	96.9	58.9	59.8
20. Unit Availability Factor	96.9	58.9	59.8
21. Unit Capacity Factor (MDC Net)	94.5	53.9	53.1
22. Unit Capacity Factor (DER Net)	91.3	52.1	51.3
23. Unit Forced Outage Rate	3.2	23.2	33.0

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): PORV MODS - 3/15/80; REFUELING - 4/14/80.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
79-26	12/01/79	S	50.2	B	5		HC	HTEXCH	POWER REDUCTION TO PERMIT CONDENSER WATERBOX CLEANING.
79-27	12/14/79	S	14.5	B	5		HC	HTEXCH	POWER REDUCTION TO PERMIT CONDENSER WATERBOX CLEANING.
79-28	12/21/79	F	23.9	A	3		IF	INSTRU	ICS SAW AN INCORRECT SIGNAL AND TRIED TO INCREASE FW TO MATCH. THE INCREASED FW BEGAN LOWERING RC PRESSURE SO FW WAS PLACED IN HAND AND DRIVEN DOWN. RC PRESSURE BEGAN TO RECOVER AND FW WAS PLACED BACK IN AUTO. RC PRESSURE CONTINUED TO INCREASE BUT FW DID NOT RECOVER AND THE UNIT TRIPPED ON HIGH PRESSURE.

SUMMARY

OPERATION WAS ROUTINE DURING THE REPORT PERIOD, WITH TWO LOAD REDUCTIONS FOR CONDENSER WATERBOX CLEANING. ON 12/21/79, THE UNIT TRIPPED ON HIGH PRESSURE.

LEGEND

[1] Type:
F–Forced
S–Scheduled

[2] Reason:
A–Equipment Failure
B–Maintenance or Test
C–Refueling
D–Regulatory Restriction
E–Operator Training & License Examination
F–Administrative
G–Operational Error
H–Other

[3] Method:
1–Manual
2–Manual Scram
3–Automatic Scram
4–Continuations
5–Load Reduction
9–Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG 0161)

[5] Exhibit H:
Same source.

NRC POWER RESTRICTIONS: NONE

DAVIS-BESSE 1

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Ohio/Ottawa 21 miles E of Toledo
2.	Electric Reliability Council	ECAR
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2772
5.	Design Electrical Rating (MWE Net)	906
6.	Date of Initial Criticality	August 12, 1977
7.	Date of Commercial Operation	11/21/77 (25%); 12/19/77 (40%) 01/23/78 (75%); 07/31/78 (100%)
8.	Condenser Cooling Method	Cooling Tower
9.	Condenser Cooling Water Source	Lake Erie

Utility & Contractor Information

10.	Licensee	Toledo Edison Co.
11.	Corporate Address	Edison Plaza, 300 Madison Avenue Toledo, Ohio 43652
12.	Corporate Contact	R. P. Crouse, V. P., Nuclear (419)259-5221
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	B&W
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Senior Inspector	R. F, Warnick
19.	IE Principal Inspector	L. Reyes
20.	Licensing Project Manager	D. Garner
21.	Docket Number	50-346
22.	License Number/Date of Issue	NPF-3, 4/22/77
23.	Public Document Room	Ida Rupp Public Library 310 Madison Street Port Clinton, OH 43452

INSPECTION STATUS

ENFORCEMENT STATUS

TECHNICAL SPECIFICATION 3.3.2.1 REQUIRES, IN PART, THAT THE EMERGENCY DIESEL GENERATOR SEQUENCE LOGIC CHANNELS BE OPERABLE WHEN THE REACTOR COOLANT SYSTEM IS ABOVE 200 DEGREES F. THE OPERABILITY OF THE SEQUENCE LOGIC CHANNELS DEPENDS UPON THE OPERABILITY OF THE ESSENTIAL BUS FEEDER BREAKER TRIP (90%) UNDERVOLTAGE SENSING RELAYS DESCRIBED IN TECHNICAL SPECIFICATION TABLE 3.3-4. CONTRARY TO THE ABOVE, ON NOVEMBER 4, 1978, DURING THE PERFORMANCE OF THE NATURAL CIRCULATION TEST AND ON SEVERAL OTHER OCCASIONS PRIOR TO DECEMBER 1978, CERTAIN SEQUENCE LOGIC CHANNELS WERE RENDERED INOPERABLE FOR SHORT PERIODS OF TIME WHEN THE REACTOR COOLANT SYSTEM AVERAGE TEMPERATURE WAS ABOVE 200 DEGREES F. THIS WAS ACCOMPLISHED BY DISABLING THE 90% UNDERVOLTAGE SENSING RELAYS WHEN STARTING A REACTOR COOLANT OR CIRCULATING WATER PUMP.
(7928 INFRACTION)

TECHNICAL SPECIFICATION 6.5.1.6.E REQUIRES THE STATION REVIEW BOARD TO INVESTIGATE ALL VIOLATIONS OF TECHNICAL SPECIFICATIONS INCLUDING PREPARATION AND FORWARDING REPORTS COVERING EVALUATION AND RECOMMENDATIONS TO PREVENT RECURRENCE TO THE VICE PRESIDENT - ENERGY SUPPLY AND TO THE CHAIRMAN OF THE COMPANY NUCLEAR REVIEW BOARD. CONTRARY TO THE ABOVE, THE STATION REVIEW BOARD DID NOT INVESTIGATE THE VIOLATIONS OF TECHNICAL SPECIFICATION AND PREPARE A REPORT FOR THE ITEMS CONTAINED IN IE INSPECTION REPORT NO. 50-346/79-04 DATED MARCH 28, 1979.
(7928 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

- The unit was shutdown December 3, 1979 to replace a leaking RDT in RCS hot leg and remained down for the month to replace damaged pipe bosses for the RTD's.

LAST IE SITE INSPECTION DATE: August 13-17, 27-31; September 10-14 and 24-28, 1979

INSPECTION REPORT NO: 79-28

SUMMARY

INSPECTION SUMMARY

- No inspections conducted.

REPORTS RECEIVED FROM LICENSEE

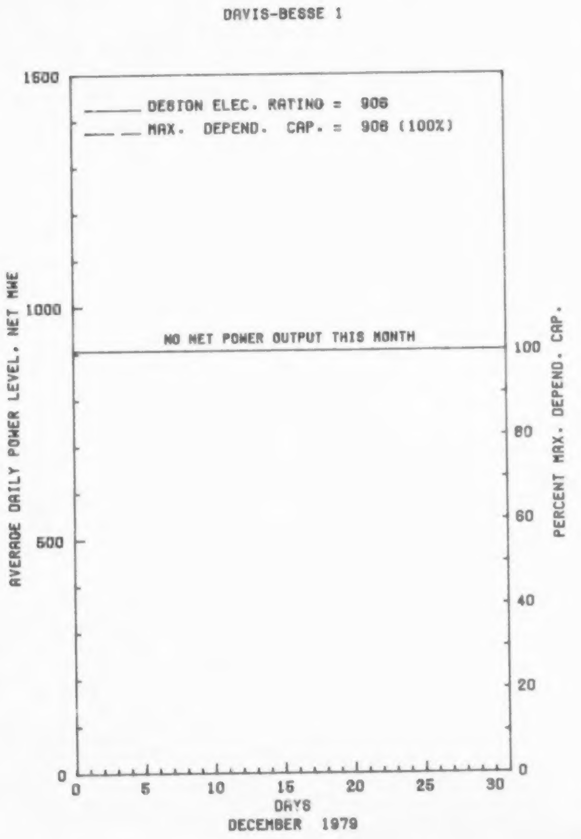
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-107/ 03L-0	11/05/79	12/04/79	30 Day LER	Loss of essential instrumentation distribution panel Y2.
79-108/ 03L-0	11/08/79	12/06/79	30 Day LER	DHR pump 1 failed to start remotely and failed to start locally at the breaker.
79-109/ 03L-0	11/13/79	12/11/79	30 Day LER	Couch relay contacts were driving high inductive loads in nuclear safety related circuits.
79-110/ 03L-0	11/12/79	12/11/79	30 Day LER	High pressure injection (HPI) failed to start.
79-111/ 03L-0	11/16/79	12/11/79	30 Day LER	Accelerometer traces could not be discerned in recorded tape due to excessive noise.

REVIEWED BY OMPA LOEB: P E Bobe DATE: 2/11/80

REVIEWED BY NRR DOR: Samuel J Gaines DATE: 2/13/80

DAVIS-BESSE 1

AVERAGE POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-346
2. Utility Contact: E. Caba (419) 259-5000 x236
3. Licensed Thermal Power (MWt): 2772
4. Nameplate Rating (Gross MWe): 1069 x 0.9 = 962
5. Design Electrical Rating (Net MWe): 906
6. Maximum Dependable Capacity (Gross MWe): TO BE DETERMINED
7. Maximum Dependable Capacity (Net MWe): TO BE DETERMINED
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	8,760.0	20,525.0
12. Hours Reactor Was Critical	29.1	4,332.3	10,964.2
13. Reactor Reserve Shutdown Hours	.0	2,085.5	2,875.8
14. Hours Generator On-Line	.0	4,141.6	9,874.7
15. Unit Reserve Shutdown Hours	.0	1,728.2	1,728.2
16. Gross Thermal Energy (MWH)	806	10,011,937	20,199,502
17. Gross Electrical Energy (MWH)	0	3,334,756	6,723,511
18. Net Electrical Energy (MWH)	0	3,129,118	6,170,578
19. Unit Service Factor	.0	47.3	49.5
20. Unit Availability Factor	.0	67.0	58.8
21. Unit Capacity Factor (MDC Net)	.0	39.4	36.2
22. Unit Capacity Factor (DER Net)	.0	39.4	36.2
23. Unit Forced Outage Rate	100.0	28.5	27.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUELING OUTAGE, MARCH 1980 - 12 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: 01/05/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
21	11/30/79	S	11.4	B	4				MAINTENANCE OUTAGE DUE TO A LOW BEARING OIL LEVEL ALARM ON REACTOR COOLANT PUMP 1-2.
21A	11/30/79	F	732.6	B	4				MAINTENANCE OUTAGE DUE TO A LOW BEARING OIL LEVEL ALARM ON REACTOR COOLANT PUMP 1-2.

SUMMARY

THE UNIT REMAINED SHUTDOWN THROUGHOUT THE REPORT PERIOD. CONTINUING THE 11/30/79 MAINTENANCE OUTAGE DUE TO A LOW BEARING OIL LEVEL ALARM ON REACTOR COOLANT PUMP 1-2.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

best copy available

DRESDEN 1

FACILITY DATA

Facility Description

1. Location (state/county) (nearest population center): Illinois/Grundy
9 miles E of Morris
2. Electric Reliability Council: Main
3. Type of Reactor: BWR
4. Power Level (MWT): 700
5. Design Electrical Rating (MWE Net): 200
6. Date of Initial Criticality: October 15, 1959
7. Date of Commercial Operation: July 4, 1960
8. Condenser Cooling Method: Once Through
9. Condenser Cooling Water Source: Kankakee River

Utility & Contractor Information

10. Licensee: Commonwealth Edison Co.
11. Corporate Address: P. O. Box 767, Chicago, Illinois 60690
12. Corporate Contact: C. Reed
13. Architect/Engineer: Bechtel
14. Nuclear Steam Supply System: GE
15. Constructor: Bechtel

Regulatory Information

16. IE Region responsible: III
17. IE Branch Chief: R. Heishman
18. IE Senior Inspector: R. Spessard
19. IE Resident Inspector: J. L. Barker (815)-942-9267
20. Licensing Project Manager: S. J. Nowicki
21. Docket Number: 50-010
22. License Number/Date of Issue: DPR-2 issued 9/28/59
23. Public Document Room: Morris Public Library
604 Liberty Street
Morris, Illinois 60451

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

Unit 1 shutdown on October 31, 1978 for an extended outage to upgrade ECCS, chemical cleaning and refueling. Outage expected to last a minimum of 18 months.

+LAST IE SITE INSPECTION DATE: October 1 through November 2, 1979

+ INSPECTION REPORT NO: 79-22

SUMMARY

INSPECTION SUMMARY

+ Inspection on October 9-12, 15-18 and 31, (79-21): Included a review of the Security Plan, Security Program Audit; Security organization; Protected Area Physical Barriers; Vital Area Physical Barriers; Lighting; Access Control (Identification, Authorization and Badging); Access Control (Search); Access Control (Escorting); Access Control (Vital Areas); Alarm Stations; Detection Aids; Assessment Aids; Communications; Testing and Maintenance; Compensatory Measures; Power Supply; Response; Locks, Keys and Combinations; Followup on Previous items of noncompliance; and Followup on IE Bulletin No. 79-16. The inspection involved 107 inspector hours onsite by two NRC inspectors. The inspection began during the back shift period; 12 inspector hours were accomplished during offshift periods. The licensee was found to be in compliance with NRC requirements within the areas examined except for the following items: Physical Barriers/Protected Area - The licensee failed to maintain the isolation zone of the protected area; Physical Barriers/Vital Area - The vital area personnel doors required modifications which would make them resistant to attack; Access Controls (Escort) - The licensee failed to control a licensee designated vehicle; Communications - The door to the telephone relay switching room was found unsecured and unattended; Testing and Maintenance - The licensee failed to test the metal detectors utilized for ingress searching according to the manufacturer's instructions. Two items of concern were noted. These items, which related to the licensee's implementation schedule of new security systems and the area of vital area authorization, were discussed in a meeting with corporate management on October 31, 1979. This meeting involved six hours by four NRC Region III personnel. The licensee's response to IE Bulletin No. 79-16 is considered an outstanding item and is being referred to IE for consideration.

+ Inspection on October 1 through November 2, (79-22): Routine, unannounced resident inspection of maintenance activities; plant operations; physical protection-security organization, physical barriers, access control (identification, authorization, badging, search and escorting), and communications; and followup on licensee event reports. The inspection involved 121 inspector hours onsite by one NRC inspector. Of the seven areas inspected, no items of noncompliance or deviations were identified.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

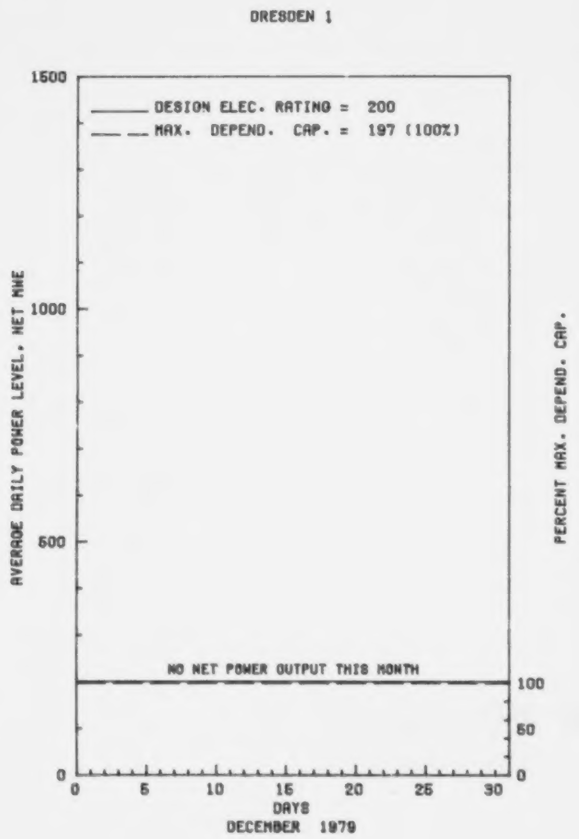
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
NONE				

REVIEWED BY OMPA LOEB: DATE: 7/11/80

REVIEWED BY NRR DOR: S J Nowicki DATE: 7/12/80

DRESDEN 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-010
2. Utility Contact: S. A. Cieszkiewicz (815) 942-2920
3. Licensed Thermal Power (MWt): 700
4. Nameplate Rating (Gross MWe): 245.5 x 0.85 = 209
5. Design Electrical Rating (Net MWe): 200
6. Maximum Dependable Capacity (Gross MWe): 205
7. Maximum Dependable Capacity (Net MWe): 197
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	8,760.0	172,800.0
12. Hours Reactor Was Critical	.0	.0	113,153.9
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	.0	.0	111,196.2
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	0	56,014,747
17. Gross Electrical Energy (MWH)	0	1	16,799,254
18. Net Electrical Energy (MWH)	-742	-13,047	15,809,827
19. Unit Service Factor	.0	.0	64.3
20. Unit Availability Factor	.0	.0	64.3
21. Unit Capacity Factor (MDC Net)	.0	.0	46.4
22. Unit Capacity Factor (DER Net)	.0	.0	45.7
23. Unit Forced Outage Rate	.0	.0	11.2

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: NO REPORT

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
8	10/31/79	S	744.0	C	4				REFUEL AND CHEMICAL CLEANING OUTAGE CONTINUES.

SUMMARY

THE UNIT REMAINED SHUTDOWN THROUGHOUT THE REPORT PERIOD FOR REFUELING AND CHEMICAL CLEANING.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

DRESDEN 2

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Illinois/Grundy 9 miles E of Morris
2.	Electric Reliability Council	Main
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2527
5.	Design Electrical Rating (MWE Net)	794
6.	Date of Initial Criticality	January 7, 1970
7.	Date of Commercial Operation	JULY, 1970
8.	Condenser Cooling Method	Cooling Lake
9.	Condenser Cooling Water Source	Kankakee and Des Plaines Rivers

Utility & Contractor Information

10.	Licensee	Commonwealth Edison Co.
11.	Corporate Address	P. O. Box 767 Chicago, Illinois 60690
12.	Corporate Contact	C. Reed
13.	Architect/Engineer	Sargent & Lundy
14.	Nuclear Steam Supply System	GE
15.	Constructor	United Engineers & Constructors, Inc.

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Senior Inspector	R. Spessard
19.	IE Resident Inspector	J. L. Barker (815)-942-9267
20.	Licensing Project Manager	S. J. Nowicki
21.	Docket Number	50-237
22.	License Number/Date of Issue	DPR-19 issued 12/22/69
23.	Public Document Room	Morris Public Library 604 Liberty Street Morris, Illinois 60451

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

Unit operating routinely.

+LAST IE SITE INSPECTION DATE: October 1 through November 2, 1979

+ INSPECTION REPORT NO: 79-26

SUMMARY

INSPECTION SUMMARY

+ Inspection on October 9-12, 15-18 and 31, (79-24): Included a review of the Security Plan, Security Program Audit; Security organization; Protected Area Physical Barriers; Vital Area Physical Barriers; Lighting; Access Control (Identification, Authorization and Badging); Access Control (Search); Access Control (Escorting); Access Control (Vital Areas); Alarm Stations; Detection Aids; Assessment Aids; Communications; Testing and Maintenance; Compensatory Measures; Power Supply; Response; Locks, Keys and Combinations; Followup on Previous items of noncompliance; and Followup on IE Bulletin No. 79-16. The inspection involved 107 inspector hours onsite by two NRC inspectors. The inspection began during the back shift period; 12 inspector hours were accomplished during offshift periods. The licensee was found to be in compliance with NRC requirements within the areas examined except for the following items: Physical Barriers/Protected Area - The licensee failed to maintain the isolation zone of the protected area; Physical Barriers/Vital Area - The vital area personnel doors required modifications which would make them resistant to attack; Access Controls (Escort) - The licensee failed to control a licensee designated vehicle; Communications - The door to the telephone relay switching room was found unsecured and unattended; Testing and Maintenance - The licensee failed to test the metal detectors utilized for ingress searching according to the manufacturer's instructions. Two items of concern were noted. These items, which related to the licensee's implementation schedule of new security systems and the area of vital area authorization, were discussed in a meeting with corporate management on October 31, 1979. This meeting involved six hours by four NRC Region III personnel. The licensee's response to IE Bulletin No. 79-16 is considered an outstanding item and is being referred to IE for consideration.

+ Inspection on October 1 through November 2, (79-26): Routine, unannounced resident inspection of maintenance activities; plant operations; physical protection-security organization, physical barriers, access control (identification, authorization, badging, search and escorting), and communications; and followup on licensee event reports. The inspection involved 121 inspector hours onsite by one NRC inspector. Of the seven areas inspected, no items of noncompliance or deviations were identified.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

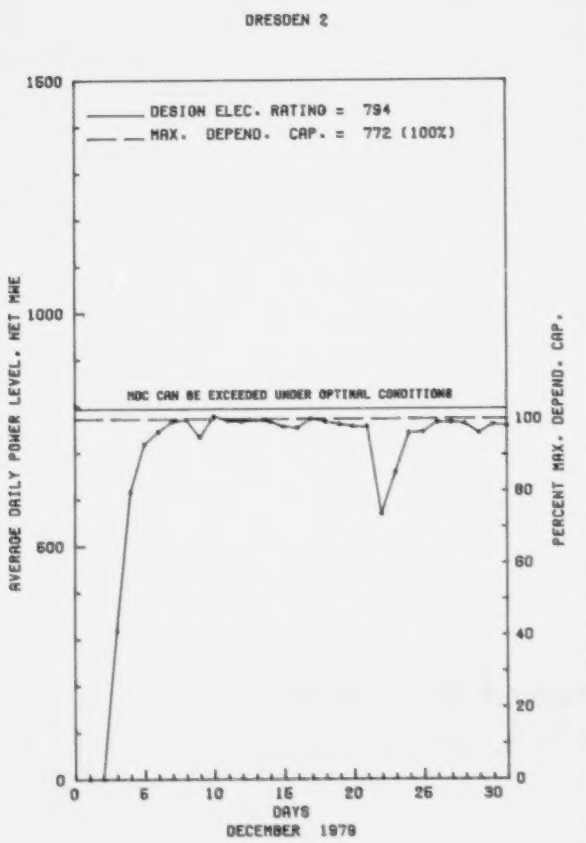
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-60/03L-0	10/30/79	11/28/79	30 Day LER	Failure to perform surveillance on testable valves in fire suppression water flow path as specified in TS.
79-61/03L-0	11/09/79	12/06/79	30 Day LER	Cardox storage tank pressure found less than TS limits.
79-62/03L-0	11/10/79	12/06/79	30 Day LER	Failure of a SBGT System.
79-63/03L-0	11/14/79	11/30/79	30 Day LER	SRM Channel 23 rod block trip in excess of TS limits.

REVIEWED BY OMPA LOEB: DATE: 2/11/80

REVIEWED BY NRR DOR: S J Nowicki DATE: 2/12/80

DRESDEN 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-237
2. Utility Contact: S. A. Cieszkiewicz (815) 942-2920 x263
3. Licensed Thermal Power (MWt): 2527
4. Nameplate Rating (Gross MWe): 920 x 0.9 = 828
5. Design Electrical Rating (Net MWe): 794
6. Maximum Dependable Capacity (Gross MWe): 812
7. Maximum Dependable Capacity (Net MWe): 772
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	8,760.0	84,456.0
12. Hours Reactor Was Critical	718.3	7,291.3	64,736.7
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	690.9	7,145.7	61,262.7
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,640,178	16,169,192	121,641,832
17. Gross Electrical Energy (MWH)	529,007	5,211,897	39,006,621
18. Net Electrical Energy (MWH)	503,812	4,939,630	36,888,112
19. Unit Service Factor	92.9	81.6	72.5
20. Unit Availability Factor	92.9	81.6	72.5
21. Unit Capacity Factor (MDC Net)	87.7	73.0	56.6
22. Unit Capacity Factor (DER Net)	85.3	71.0	55.0
23. Unit Forced Outage Rate	.0	3.2	13.4

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
14	11/30/79	S	53.1	A	4				REPAIR LEAK ON MOISTURE SEPARATOR LINE. ALSO, TMI MODIFICATIONS - ACOUSTIC MONITORS INSTALLED ON SAFETY AND RELIEF VALVE DISCHARGE LINES.

SUMMARY

OPERATION WAS ROUTINE DURING THE REPORT PERIOD, FOLLOWING AN OUTAGE FROM 12/01/79 TO 12/03/79 TO REPAIR A LEAK ON THE MOISTURE SEPARATOR LINE AND INSTALL ACOUSTIC MONITORS ON SAFETY AND RELIEF VALVE DISCHARGE LINES.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

DRESDEN 3

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Illinois/Grundy 9 miles E of Morris
2.	Electric Reliability Council	Main
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2527
5.	Design Electrical Rating (MWE Net)	794
6.	Date of Initial Criticality	January 31, 1971
7.	Date of Commercial Operation	November 16, 1971
8.	Condenser Cooling Method	Cooling Lake
9.	Condenser Cooling Water Source	Kankakee and Des Plaines Rivers

Utility & Contractor Information

10.	Licensee	Commonwealth Edison Co.
11.	Corporate Address	P. O. Box 767 Chicago, Illinois 60690
12.	Corporate Contact	D. L. Peoples
13.	Architect/Engineer	Sargent & Lundy
14.	Nuclear Steam Supply System	GE
15.	Constructor	United Engineers & Constructors, Inc.

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Senior Inspector	R. Spessard
19.	IE Resident Inspector	J. L. Barker 815-942-9267
20.	Licensing Project Manager	R. Bevan
21.	Docket Number	50-249
22.	License Number/Date of Issue	DPR-25 issued 1/12/71
23.	Public Document Room	Morris Public Library 604 Liberty Street Morris, Illinois 60451

INSPECTION STATUS

ENFORCEMENT STATUS

UNIT 3 TECHNICAL SPECIFICATIONS. SECTION 6.2.A REQUIRES THAT DETAILED PROCEDURES INVOLVING OPERATION OF COMPONENTS INVOLVING NUCLEAR SAFETY OF THE FACILITY BE PREPARED, APPROVED, AND ADHERED TO. LICENSEE PROCEDURE DOP 2000-2-M1, "A AND B WASTE COLLECTOR FILTERS MANUAL VALVE LINE-UP CHECKLISTS," DELINEATES THE PROPER VALVE LINE-UP FOR A AND B WASTE COLLECTOR SYSTEM OPERATIONS. CONTRARY TO THE ABOVE, ON OCTOBER 25, 1979, A LICENSEE EMPLOYEE (AN UNQUALIFIED EQUIPMENT ATTENDANT) FAILED TO MAKE VALVING EVOLUTIONS IN ACCORDANCE WITH DOP 2000-2-M1, WHICH RESULTED IN THE OVERFILLING OF THE UNIT 3 SPENT FUEL POOL AND SUBSEQUENT CONTAMINATION OF THE UNIT 3 REACTOR BUILDING WITH LOW LEVEL RADIOACTIVE MATERIAL.
(7924 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

The unit is operating at 70% power (Administrative Restriction Imposed by licensee) due to increase in offgas activity and 7 x 7 fuel assembly cladding degradation.

*LAST IE SITE INSPECTION DATE: October 1 through November 2, 1979

* INSPECTION REPORT NO: 79-24

SUMMARY

INSPECTION SUMMARY

* Inspection on October 9-12, 15-18 and 31, (79-22): Included a review of the Security Plan, Security Program Audit; Security organization; Protected Area Physical Barriers; Vital Area Physical Barriers; Lighting; Access Control (Identification, Authorization and Badging); Access Control (Search); Access Control (Escorting); Access Control (Vital Areas); Alarm Stations; Detection Aids; Assessment Aids; Communications; Testing and Maintenance; Compensatory Measures; Power Supply; Response; Locks, Keys and Combinations; Followup on Previous items of noncompliance; and Followup on IE Bulletin No. 79-16. The inspection involved 107 inspector hours onsite by two NRC inspectors. The inspection began during the back shift period; 12 inspector hours were accomplished during offshift periods. The licensee was found to be in compliance with NRC requirements within the areas examined except for the following items: Physical Barriers/Protected Area - The licensee failed to maintain the isolation zone of the protected area; Physical Barriers/Vital Area - The vital area personnel doors required modifications which would make them resistant to attack; Access Controls (Escort) - The licensee failed to control a licensee designated vehicle; Communications - The door to the telephone relay switching room was found unsecured and unattended; Testing and Maintenance - The licensee failed to test the metal detectors utilized for ingress searching according to the manufacturer's instructions. Two items of concern were noted. These items, which related to the licensee's implementation schedule of new security systems and the area of vital area authorization, were discussed in a meeting with corporate management on October 31, 1979. This meeting involved six hours by four NRC Region III personnel. The licensee's response to IE Bulletin No. 79-16 is considered an outstanding item and is being referred to IE for consideration.

-* Inspection on October 1 through November 2, (79-24): Routine, unannounced resident inspection of maintenance activities; plant operations; physical protection-security organization, physical barriers, access control (identification, authorization, badging, search and escorting), and communications; and followup on licensee event reports. The inspection involved 121 inspector hours onsite by one NRC inspector. Of the seven areas inspected, there were no items of noncompliance identified in six areas. One item of noncompliance (failure to follow procedures) was identified in one area.

NRC POWER RESTRICTIONS: NONE

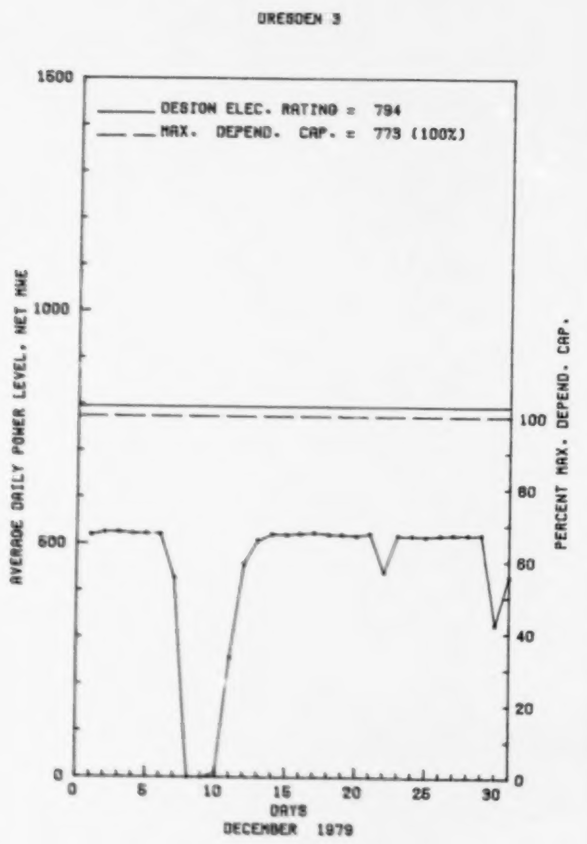
REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-29/ 03L-0	11/01/79	-11/28/79	30 Day LER	IRM Channel 11 tripped in excess of TS limits.
79-30/ 03L-0	11/10/79	12/06/79	30 Day LER	Failure of RPIS on CRD H-9.
79-31/ 03L-0	11/17/79	12/07/79	30 Day LER	MSIV 3-203-2D closure less than TS limits.
79-32/ 03L-0	11/20/79	12/10/79	30 Day LER	Pinhole leak found on 3B Reactor feed pump.

REVIEWED BY OMPA LOEB: DATE: 2/11/80
REVIEWED BY NRR DOR: Dick Clark DATE: 02-14-80

DRESDEN 3

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-249
2. Utility Contact: S. A. Cieszkiewicz (815) 942-2920 x263
3. Licensed Thermal Power (MWt): 2527
4. Nameplate Rating (Gross MWe): 920 x 0.9 = 828
5. Design Electrical Rating (Net MWe): 794
6. Maximum Dependable Capacity (Gross MWe): 812
7. Maximum Dependable Capacity (Net MWe): 773
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): 550
10. Reasons for Restrictions, If Any: ADMIN. DERATING-AIR EJECT. EVAL.

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	8,760.0	74,041.0
12. Hours Reactor Was Critical	680.2	6,056.7	55,765.4
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	665.9	5,933.8	53,324.6
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,092,724	11,512,900	102,271,668
17. Gross Electrical Energy (MWH)	350,627	3,678,516	33,429,674
18. Net Electrical Energy (MWH)	331,222	3,475,813	31,693,291
19. Unit Service Factor	89.5	67.7	72.0
20. Unit Availability Factor	89.5	67.7	72.0
21. Unit Capacity Factor (MDC Net)	57.5	51.3	55.4
22. Unit Capacity Factor (DER Net)	56.1	50.0	53.9
23. Unit Forced Outage Rate	1.1	31.0	16.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUEL, MARCH 17, 1980, APPROX 10 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
11	12/07/79	S	70.7	H	1				TMI MODIFICATIONS - ACOUSTIC MONITORS INSTALLED ON SAFETY VALVE DISCHARGE LINES.
12	12/10/79	F	7.4	A	3				TURBINE TRIPPED ON 3"B" MOISTURE SEPARATOR HI HI, RX SCRAMMED ON HI NEUTRON FLUX.

SUMMARY

OPERATION WAS ROUTINE AT AN ADMINISTRATIVE DERATING FOR AIR EJECTOR EVALUATIONS. THE UNIT WAS TAKEN OFF LINE FROM 12/07/79 THROUGH 12/11/79 TO INSTALL ACOUSTIC MONITORS ON SAFETY AND RELIEF VALVE DISCHARGE LINES. A TURBINE TRIP OCCURRED ON 12-10-79 ON MOISTURE SEPARATION DRAIN TANK HIGH LEVEL FOLLOWING STARTUP.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

NRC POWER RESTRICTIONS: NONE

DUANE ARNOLD

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Iowa/Linn 8 miles NW of Cedar Rapids
2.	Electric Reliability Council	N/A
3.	Type of Reactor	BWR
4.	Power Level (MWT)	1658
5.	Design Electrical Rating (MWE Net)	538
6.	Date of Initial Criticality	March 23, 1974
7.	Date of Commercial Operation	February 1, 1975
8.	Condenser Cooling Method	Cooling Towers
9.	Condenser Cooling Water Source	Cedar Rapids River Circulating Water System

Utility & Contractor Information

10.	Licensee	Iowa Electric Light and Power
11.	Corporate Address	IE Towers, P. O. Box 351 Cedar Rapids, Iowa 52406
12.	Corporate Contact	D. Arnold, President
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	GE
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Senior Inspector	R. Spessard
19.	IE Principal Inspector	G. Wright
20.	Licensing Project Manager	T. Kevern
21.	Docket Number	50-331
22.	License Number/Date of Issue	DPR-49 issued 2/22/74
23.	Public Document Room	Reference Service Cedar Rapids Public Library 428 Third Avenue, S.E. Cedar Rapids, Iowa 52401

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

Unit operating routinely.

*LAST IE SITE INSPECTION DATE: November 28-29, 1979

* INSPECTION REPORT NO: 79-30

SUMMARY

INSPECTION SUMMARY

* Inspection on October 29 through November 2, (79-28): Included a review of Security Plan; Security Program Audit; Security Organization; Protected Area Physical Barriers; Vital Area Physical Barriers; Lighting; Access Control (Identification, Authorization and Badging); Access Control (Search); Access Control (Escorting); Access Control (Vital Areas); Alarm Stations; Detection Aids; Assessment Aids; Communications; Testing and Maintenance; Compensatory Measures; Power Supply; Response; Locks, Keys, and Combinations; Records and Reports; Followup on Previous Items of Noncompliance; Followup on IE Bulletin No. 79-16. The inspection involved 84 inspector hours onsite by two NRC ispectors. The inspection began during the day shift period; 12 inspector hours were accomplished during offshift periods. The licensee was found to be in compliance with NRC requirements within the 20 areas examined during this inspection except for the following four items: Access Control (Vital Areas) - The licensee failed to provide positive access control to both an ICAA and a vital area; Security Organization - Incomplete documentation on the training and qualification records of several security guards; Access Control (Search) - One vehicle which entered the protected area was not properly searched; and Locks, Keys, and Combinations - Some keys were not properly controlled.

* Inspection on November 27-28, (79-29): Nonroutine, announced inspection of the licensee's response to IE Bulletin No. 79-19. The inspection involved 15 inspector hours onsite by one NRC inspector. No items of noncompliance or deviations were identified.

* Inspection on November 28-29, (79-30): Routine, unannounced inspection of previously identified outstanding items and follow-up to IE Bulletin No. 79-02, Pipe Support Base Plate Designs Using Concrete Expansion Anchors. The inspection involved 14 inspector hours onsite by one NRC inspector. No items of noncompliance were identified.

REPORTS RECEIVED FROM LICENSEE

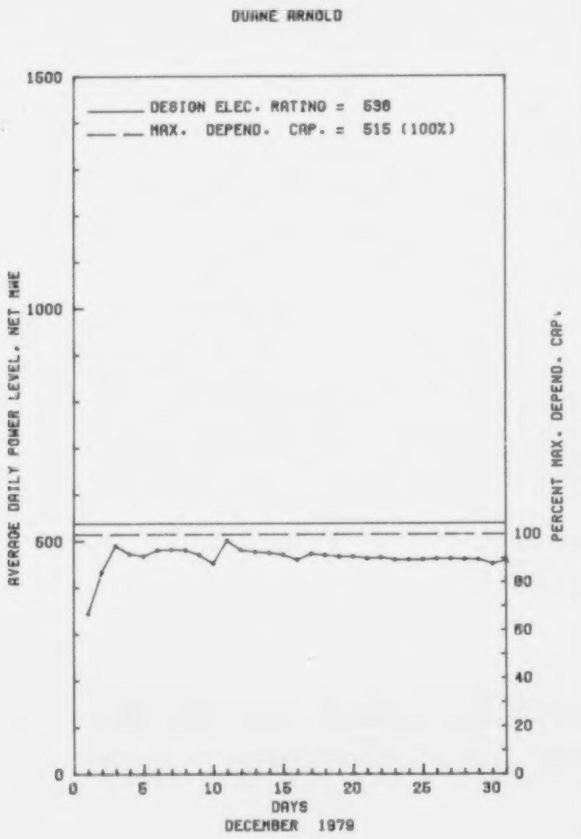
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-30/ 03L-0	11/12/79	12/10/79	30 Day LER	RCIC high flow isolation dp switch calibrator drift.
79-31/ 03L-0	11/10/79	12/10/79	30 Day LER	RBM B inoperable.
79-32/ 03L-0	11/16/79	12/12/79	30 Day LER	High drywell pressure switch out of calibration.
79-33/ 01T-0	11/28/79	12/12/79	14 Day LER	"B" O2 analyzer inoperable for suppression pool and "A" O2 analyzer unable to draw sample.

REVIEWED BY OMPA LOEB: John J. [illegible] DATE: 2-11-80

REVIEWED BY NRR DOR: [illegible] DATE: [illegible]

DUANE ARNOLD

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-331
2. Utility Contact: J. Van Sickel (319) 851-5611
3. Licensed Thermal Power (MWt): 1658
4. Nameplate Rating (Gross MWe): 663 x 0.9 = 597
5. Design Electrical Rating (Net MWe): 538
6. Maximum Dependable Capacity (Gross MWe): 545
7. Maximum Dependable Capacity (Net MWe): 515
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	8,760.0	43,080.0
12. Hours Reactor Was Critical	744.0	6,951.3	30,564.1
13. Reactor Reserve Shutdown Hours	.0	.0	[illegible]
14. Hours Generator On-Line	744.0	6,832.1	29,819.1
15 Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,045,416	9,067,920	37,129,560
17. Gross Electrical Energy (MWH)	366,935	3,086,937	12,424,366
18. Net Electrical Energy (MWH)	345,093	2,898,764	11,615,064
19. Unit Service Factor	100.0	78.0	69.2
20. Unit Availability Factor	100.0	78.0	69.2
21. Unit Capacity Factor (MDC Net)	90.1	64.3	52.4
22. Unit Capacity Factor (DER Net)	86.2	61.5	50.1
23. Unit Forced Outage Rate	.0	22.0	20.9

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUELING, FEBRUARY 9, 1980, 12 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
22	12/01/79	S	.0	F	5		RB		CONROD POWER REDUCTION TO WITHDRAW CONTROL RODS.

SUMMARY

OPERATION WAS ROUTINE IN AN END OF CYCLE COASTDOWN, WITH A FURTHER REDUCTION FOR CONTROL ROD ADJUSTMENT.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

FARLEY 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Alabama/Houston 24 miles SE of Dothan
2.	Electric Reliability Council	SERC
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2652
5.	Design Electrical Rating (MWE Net)	829
6.	Date of Initial Criticality	August 9, 1977
7.	Date of Commercial Operation	December 1, 1977
8.	Condenser Cooling Method	Once Through With Forced Draft Cooling Towers
9.	Condenser Cooling Water Source	Chattahoochee River

Utility & Contractor Information

10.	Licensee	Alabama Power Company
11.	Corporate Address	600 North 18th Street Birmingham, Alabama
12.	Corporate Contact	F. L. Clayton, Jr. Senior Vice President (205-323-5341)
13.	Architect/Engineer	Bechtel/SSI
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Daniel

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. Lewis
18.	IE Senior Inspector	R. D. Martin
19.	IE Principal Inspector	W. H. Bradford
20.	Licensing Project Manager	E. A. Reeves
21.	Docket Number	50-348
22.	License Number/Date of Issue	NPF-2, 6/25/77
23.	Public Document Room	G. S. Houston Memorial Library 212 W. Berdeshaw Street Dothan, Alabama 36301

INSPECTION STATUS

ENFORCEMENT STATUS

AS REQUIRED BY TECHNICAL SPECIFICATION 4.3-3.2.A, A MONTHLY CHANNEL FUNCTIONAL TEST OF FUEL STORAGE POOL AREA PROCESS MONITORS A & B SHALL BE DONE WHEN IRRADIATED FUEL IS IN THE STORAGE POOL. CONTRARY TO THE ABOVE, A REVIEW OF SURVEILLANCE TEST PROCEDURE FNP-1-STP-43.0, FUEL STORAGE POOL PROCESS MONITOR A & B - MONTHLY FUNCTIONAL TEST, FOR THE PERIOD APRIL TO SEPTEMBER 1979, INDICATED THAT THE FUNCTIONAL TEST WAS NOT PERFORMED FOR THE MONTH OF MAY 1979.
(7936 DEFICIENCY)

AS REQUIRED BY TECHNICAL SPECIFICATION 6.8.1.C, WRITTEN PROCEDURES SHALL BE ESTABLISHED, IMPLEMENTED AND MAINTAINED, COVERING THE ACTIVITIES CONCERNING SURVEILLANCE AND TEST OF SAFETY RELATED EQUIPMENT. SECTION 3.2.9 OF FARLEY NUCLEAR PLANT ADMINISTRATIVE PROCEDURE AP-5, SURVEILLANCE PROGRAM ADMINISTRATIVE CONTROL, REQUIRES THAT RESULTS OF SURVEILLANCE TEST SHALL BE REVIEWED BY EACH PLANT STAFF GROUP PERFORMING THE TESTS TO DETERMINE THAT ALL REQUIRED DATA HAS BEEN PROPERLY RECORDED AND SIGNED-OFF OR INITIALED WHERE REQUIRED. CONTRARY TO THE ABOVE, THE REQUIRED SIGNATURES WERE OMITTED A TOTAL OF EIGHT DIFFERENT PLACES THROUGHOUT STP-33.0 (SOLID STATE PROTECTION SYSTEM TRAIN A(B) OPERABILITY TEST) WHICH WAS PERFORMED ON 12/9/78 AND STP-7.0 (QUADRANT POWER TILT RATIO CALCULATION) WHICH WAS PERFORMED ON 1/22/79 AND 2/18/79 RESPECTIVELY. ALTHOUGH THESE DOCUMENTS HAD RECEIVED TWO LEVELS OF REVIEW, THE OMISSION HAD NOT BEEN IDENTIFIED.
(7937 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

Routine operations – Unit operating at full power at the end of December.

+ LAST IE SITE INSPECTION DATE: DECEMBER 3-29, 1979

+ INSPECTION REPORT NO: 50-348/79-45

SUMMARY

INSPECTION SUMMARY

+ INSPECTION NOVEMBER 7-8 (79-35): THIS SPECIAL, ANNOUNCED INSPECTION INVOLVED 28 INSPECTOR-HOURS AT THE GENERAL OFFICES IN REVIEW OF PROCEDURES AND CONTROLS ADOPTED BY ALABAMA POWER COMPANY (APCO) TO IMPLEMENT THE REQUIREMENTS OF 10 CFR PART 21. ONE ITEM OF NONCOMPLIANCE INVOLVING ADEQUACY AND IMPLEMENTATION OF PROCEDURES TO IMPLEMENT THE REQUIREMENTS OF 10 CFR 21 WAS IDENTIFIED (INFRACTION - PART 21 EVALUATION).

INSPECTION NOVEMBER 5-9 AND 13-16 (79-36): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 113 INSPECTOR-HOURS ON SITE IN THE AREAS OF COORDINATION WITH OFFSITE SUPPORT AGENCIES: EMERGENCY FACILITIES, EQUIPMENT AND PROCEDURES; MEANS FOR DETERMINING A RELEASE; EMERGENCY TRAINING; EMERGENCY DRILLS; FIRE BRIGADE ORGANIZATION AND TRAINING; EMERGENCY ORGANIZATION; IE BULLETIN 79-18; FOLLOWUP ON PREVIOUS INSPECTION FINDINGS AND FOLLOWUP ON MEDICAL EMERGENCY OF APRIL 6, 1979. OF THE TEN AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN NINE AREAS; ONE APPARENT ITEM OF NONCOMPLIANCE WAS FOUND IN ONE AREA (DEFICIENCY - FAILURE TO PERFORM MONTHLY CHANNEL FUNCTIONAL TEST ON PROCESS MONITORS R-25 A&B FOR THE FUEL STORAGE POOL AREA AS REQUIRED BY TECHNICAL SPECIFICATIONS 4.3.3).

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-043/ 01T-0	11/07/79	11/07/79	14 DAY LER	FIRE PROTECTION TANK LEVEL BELOW TS LIMIT OF 12500 GALLONS.
79-046/ 03L-0	11/03/79	11/20/79	30 DAY LER	CONDENSATE TANK 5000 GAL BELOW TS LIMIT.

INSPECTION NOVEMBER 13-16 (79-37): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 60 INSPECTOR-HOURS ON SITE IN THE AREAS OF OPERATIONS REVIEW; REVIEW OF UNIT NO. 2, PREOPERATIONAL STARTUP TEST PROGRAM; CALIBRATION OF INPLANT TECHNICAL SPECIFICATION REQUIRED INSTRUMENTATION; CALIBRATION OF MEASURING AND TEST EQUIPMENT USED ON SAFETY RELATED SYSTEMS; AND SURVEILLANCE OF INPLANT TECHNICAL SPECIFICATION REQUIRED SYSTEMS. OF THE FIVE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN FOUR AREAS; ONE APPARENT ITEM OF NONCOMPLIANCE WAS FOUND IN ONE AREA (DEFICIENCY - FAILURE TO FOLLOW PROCEDURES ON SURVEILLANCE PROGRAM ADMINISTRATIVE CONTROL).

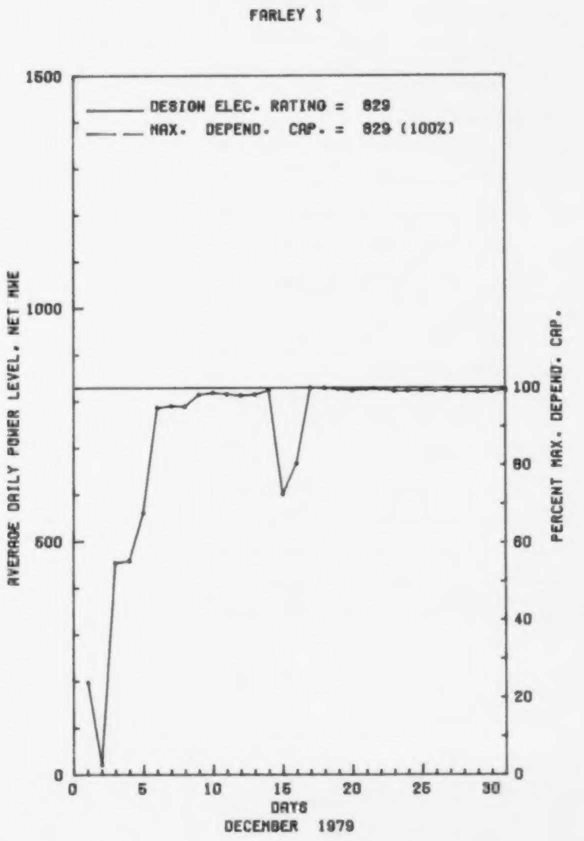
INSPECTION NOVEMBER 29-30 (79-40): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 16 INSPECTOR-HOURS ON SITE IN THE AREAS OF APPENDIX B NONRADIOLOGICAL ENVIRONMENTAL PROTECTION REQUIREMENTS INCLUDING MANAGEMENT CONTROLS, STATUS OF AQUATIC BIOLOGICAL SURVEILLANCE, REVIEW OF ANNUAL ENVIRONMENTAL REPORT, AND REVIEW OF REPORTING REQUIREMENTS. OF THE FOUR AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

REVIEWED BY OMPA LOEB: T.C. CINTULA **DATE:** 2011080

REVIEWED BY NRR DOR: Reeves **DATE:** 2-13-80

FARLEY 1

AVERAGE POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-348
2. Utility Contact: W. G. Hairston,III (205) 899-5156
3. Licensed Thermal Power (MWt): 2652
4. Nameplate Rating (Gross MWe): 1045 x 0.85 = 888
5. Design Electrical Rating (Net MWe): 829
6. Maximum Dependable Capacity (Gross MWe): 860
7. Maximum Dependable Capacity (Net MWe): 829
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	8,760.0	18,264.0
12. Hours Reactor Was Critical	744.0	2,691.7	10,935.5
13. Reactor Reserve Shutdown Hours	.0	397.8	1,405.3
14. Hours Generator On-Line	721.6	2,504.8	10,590.5
15. Unit Reserve Shutdown Hours	.0	0	.0
16. Gross Thermal Energy (MWH)	1,760,937	5,774,767	26,598,015
17. Gross Electrical Energy (MWH)	572,434	1,849,840	8,524,938
18. Net Electrical Energy (MWH)	541,938	1,743,590	8,059,416
19. Unit Service Factor	97.0	28.6	58.0
20. Unit Availability Factor	97.0	28.6	58.0
21. Unit Capacity Factor (MDC Net)	87.9	24.0	53.2
22. Unit Capacity Factor (DER Net)	87.9	24.0	53.2
23. Unit Forced Outage Rate	.4	6.1	7.6

24. Shutdowns Scheduled Over Next 6 Months (Type,Date,and Duration of Each): SURVEILLANCE TESTING - 03/14/80 - 2 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
029	11/30/79	F	3.2	G	4		HH	CKTBKR	CONTINUATION FROM 11/30/79. THE UNIT LOSS OF SGFP SUCTION PRESSURE.
030	12/01/79	F	6.7	H	5		HC	XXXXXX	UNIT WAS LIMITED TO 32% REACTOR POWER DUE TO DECREASING CONDENSER VACUUM. CAUSE UNKNOWN.
031	12/02/79	S	19.2	B	9		HH	PUMPXX	GENERATOR WAS TAKEN OFF-LINE TO ISOLATE 1B SGFP FOR REPAIRS. GENERATOR RETURNED ON-LINE.
032	12/03/79	F	7.3	B	5		HH	PIPEXX	UNIT WAS LIMITED TO 57% REACTOR POWER DUE TO LOW OIL PRESSURE ON 1A SGFP. EH FLUID LEAK WAS REPAIRED.
033	12/03/79	F	20.0	B	5		HH	PUMPXX	UNIT WAS LIMITED TO 65% REACTOR POWER WITH ONLY ONE SGFP IN SERVICE. 1B SGFP BEING REPAIRED.
034	12/04/79	F	5.7	B	5	79-062	HA	MECFUN	UNIT WAS REDUCED TO 39% REACTOR POWER DUE TO PROBLEMS WITH THE DEH VIDAR UNIT. VIDAR UNIT WAS REPAIRED.
035	12/05/79	F	16.1	B	5		HH	PUMPXX	UNIT WAS LIMITED TO 65% REACTOR POWER WITH ONLY ONE SGFP IN SERVICE. 1B SGFP BEING REPAIRED.
036	12/08/79	S	5.6	B	5		HJ	ZZZZZZ	UNIT WAS REDUCED TO 89% REACTOR POWER FOR PURPOSES OF REACTOR POWER DISTRIBUTION CONTROL.
037	12/15/79	S	10.0	B	5		HA	VALVEX	UNIT WAS REDUCED TO 65% REACTOR POWER TO COLLECT VALVE DATA FOR THE TURBINE CONTROL VALVE MANAGEMENT PROGRAM. INCORE-EXCORE NUCLEAR INSTRUMENTATION CROSS-CALIBRATION PERFORMED.
038	12/15/79	S	1.3	B	5		RC	ZZZZZZ	UNIT WAS HELD AT 75% REACTOR POWER TO RUN A FLUX MAP.
039	12/15/79	S	8.5	B	5		HA	VALVEX	UNIT WAS REDUCED TO 36% REACTOR POWER TO COLLECT VALVE DATA FOR THE TURBINE CONTROL VALVE-MANAGEMENT PROGRAM.
040	12/16/79	S	1.3	B	5		HA	VALVEX	UNIT WAS LIMITED TO 87% REACTOR POWER FOR TURBINE CONTROL VALVE TESTING.

SUMMARY

THE UNIT RETURNED TO SERVICE ON 12/01/79 FOLLOWING RECOVERY FROM AN 11/30/80 TRIP, BUT WAS LIMITED TO 32% REACTOR POWER DUE TO DECREASING CONDENSER VACUUM. AN OUTAGE OCCURRED ON 12/02/79 TO ISOLATE THE 1B STEAM GENERATOR FEEDPUMP FOR REPAIRS. THE UNIT EXPERIENCED NINE MORE LOAD REDUCTIONS DURING THE REPORT PERIOD.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

FITZPATRICK

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	New York/Oswego 8 miles NE of Oswego
2.	Electric Reliability Council	NPCC
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2436
5.	Design Electrical Rating (MWE Net)	821
6.	Date of Initial Criticality	November 17, 1974
7.	Date of Commercial Operation	July 28, 1975
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Lake Ontario

Utility & Contractor Information

10.	Licensee	Power Authority of the State of New York
11.	Corporate Address	10 Columbus Circle New York, New York 10019
12.	Corporate Contact	P. Lyon, Manager of Nuclear Operations
13.	Architect/Engineer	Stone & Webster
14.	Nuclear Steam Supply System	GE
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner (X246)
18.	IE Senior Inspector	E. McCabe (X251)
19.	IE Principal Inspector	T. F. Stetka (X276)
20.	Licensing Project Manager	P. Polk (X7872)
21.	Docket Number	50-333
22.	License Number/Date of Issue	DPR-59, 10/17/74
23.	Public Document Room	State University College of Oswego Penfield Library - Documents Oswego, NY 13126 (315)341-2323

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

Power reductions have been required due to reactor water conductivity problems caused by RWCU filter defects. Reactor thermal power is also restricted to 2 percent less than rated due to CMFLPD limit.

FACILITY ITEMS (PLANS AND PROCEDURES):

Snubber inspection and refueling outage scheduled for March, 1980.

MANAGERIAL ITEMS:

None

PLANT STATUS:

Operating at reduced power. Offgas rate at the SJAE is 3060 microcuries per second. Stack release rate is less than 50 microcuries per second.

LAST IE SITE INSPECTION DATE: 9/21/79

INSPECTION REPORT NO: 79-16

SUMMARY

INSPECTION SUMMARY

Jun 24-29 (79-08): Routine unannounced inspection of the Security Plan, Security Program Audit, Security Organization, Physical Barriers, Lighting, Access Control, Alarm Stations, Detection Aids, Assessment Aids, Communications, Testing and Maintenance, Compensatory Measures, Power Supply, Response, Locks, Keys and Combinations and Records and Reports. The inspection involved 132 inspector hours onsite by three NRC Regional inspectors. The inspection was begun during an offshift period (weekend); 44 inspection hours were performed during offshift periods. Five items of noncompliance were identified (Guards not performing all vital area duties committed to in security plan; metal detectors not operating properly; Vital Area door found unsecured; Secondary Alarm Station not equipped with audible console annunciator; Administrative record of guard responses not detailed).

Sep 10-14 and 17-20 (79-13): Routine, unannounced inspection of the QA Program; Design Change/Modification Program; Design Changes/Modifications; QA/QC Administration; Records Program; Document Control Program; and, the Offsite Support Staff. The inspection involved 71 inspector-hours onsite by two NRC inspectors and 51.5 inspector-hours at the corporate offices. Five items of noncompliance were identified (Failure to submit annual 10 CFR 50.59 report; Implementing a Technical Specification change without prior NRC approval; Failure to store single copy records per N45.2.9; Failure to take corrective action with respect to audit report findings; Failure to have drawings appropriate to the circumstances).

Sep 11-13 (79-15): Routine, unannounced inspection of Facility Organization and Operation, Measurement and Controls, Shipping and Receiving, Storage and Internal Control, Inventory, and Records and Reports. The inspection involved 45 inspector-hours onsite by two NRC inspectors. No items of noncompliance were identified.

REPORTS RECEIVED FROM LICENSEE

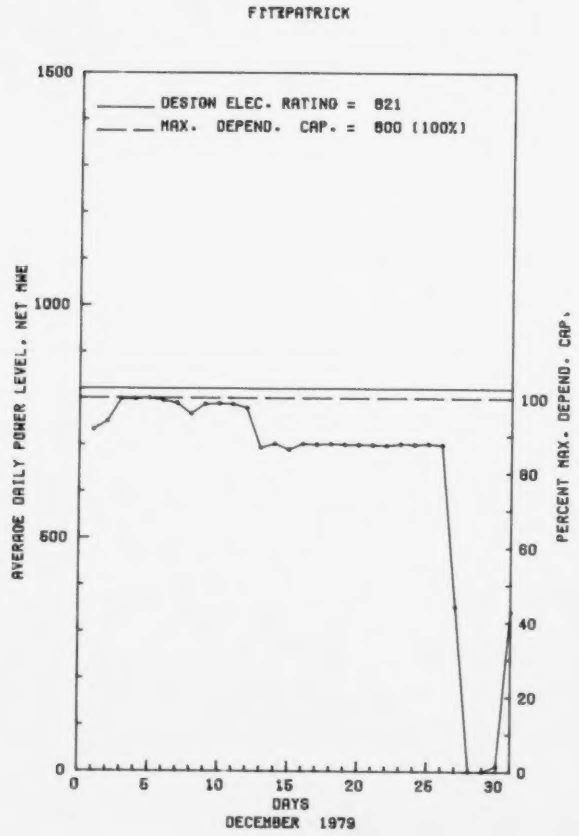
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-072/ 03L	09/12/79	10/12/79	30 Day LER	Instrument root valve positions were not verified prior to plant startup due to inadequate valve lineup procedures
79-073/ 03L	09/14/79	10/12/79	30 Day LER	Emergency diesel C tripped on overspeed during testing due to loose connection on the switch
79-075/ 03L	09/15/79	10/12/79	30 Day LER	Drywell equipment drain sump flow indicator failed to operate due to blown fuse
79-078/ 03L	10/02/79	10/30/79	30 Day LER	One reactor water level instrument was found out of calibration in the nonconservative direction
79-080/ 01T	10/17/79	10/23/79	Other	Pipe support H19-228 on spent fuel cooling system was inoperable due to piping seismic analysis
79-081/ 01T	10/18/79	10/23/79	Other	Pipe hangers on RHR and RHRSW systems found inoperable due to seismic analysis results
79-082/ 03L	10/05/79	11/02/79	30 Day LER	A lubrication line on the fire protection system required repair due to a leak
79-084/ 03L	10/09/79	11/06/79	30 Day LER	APRM channels A and D high power level scram setpoint drift
79-085/ 01T	10/24/79	11/06/79	Other	Hi-press fire pump disch snubber BFSK 65-8-H2 inop but system still operable
79-086/ 01T	10/25/79	11/07/79	Other	Seismic restraints on service water systems common return to lake piping inoperable due to shock III analysis
79-087/ 01T	10/29/79	11/07/79	Other	3 seismic supports in B RHR system inoper. repairs effected on preliminary data prior to licensee notif of inoperability one restraint in RHR SW inoperative. Being repaired
79-088/ 01T	10/31/79	11/09/79	Other	Pipe supports on fuel pool cooling system and containment vent and purge system found to be inoperative by analysis
79-089/ 01T	11/02/79	11/13/79	Other	Three pipe hangers, one SLC, one on fire protection and one on cont. vent and purge analyzed to be inoperable. only vent and purge hanger required immediate modification
79-090/ 03L	10/11/79	11/07/79	30 Day LER	HPCI system condensate storage tank suction valve disabled to allow re-routing of electrical cable
79-091/ 03L	10/20/79	11/15/79	30 Day LER	RHR pump C tripped off during surveillance testing due to improper adjustment of pump suction valve limit switches
79-092/ 03L	10/13/79	11/07/79	30 Day LER	Intermittent isolation signal to HPCI steam supply inside isolation valve caused by loose lead
79-094/ 03L	10/25/79	11/16/79	30 Day LER	Master valve on ten ton cardox system did not open properly during surveillance testing
79-095/ 03L	10/25/79	11/21/79	30 Day LER	Drywell-suppression chamber differential pressure instrument failed and had to be replaced

REVIEWED BY OMPA LOEB: John J. Rozzo DATE: 2-11-80

REVIEWED BY NRR DOR: Philip J. Polk DATE: 2/13/80

FITZPATRICK

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

ITEM 21 CALCULATED
BY USING A WEIGHTED
AVERAGE

1. Docket Number: 50-333
2. Utility Contact: E. Zufelt (315) 342-3840
3. Licensed Thermal Power (MWt): 2436
4. Nameplate Rating (Gross MWe): 981 x 0.9 = 883
5. Design Electrical Rating (Net MWe): 821
6. Maximum Dependable Capacity (Gross MWe): 830
7. Maximum Dependable Capacity (Net MWe): 800
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	8,760.0	38,833.0
12. Hours Reactor Was Critical	680.6	4,568.0	26,630.2
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	663.0	4,450.9	25,738.0
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,428,912	8,974,046	51,188,194
17. Gross Electrical Energy (MWH)	492,380	3,058,830	17,353,700
18. Net Electrical Energy (MWH)	477,050	2,964,590	16,790,575
19. Unit Service Factor	89.1	50.8	66.3
20. Unit Availability Factor	89.1	50.8	66.3
21. Unit Capacity Factor (MDC Net)	80.1	42.3	59.7
22. Unit Capacity Factor (DER Net)	78.1	41.2	52.7
23. Unit Forced Outage Rate	10.9	49.2	21.0

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): SNUBBER INSPECT & REFUEL - 04/80.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
10	12/27/79	F	81.0	H	3				TURBINE TRIP ON HIGH VIBRATION AS A RESULT OF TRANSMISSION GRID DISTURBANCE.

SUMMARY

OPERATION WAS ROUTINE DURING THE REPORT PERIOD, WITH TWO BRIEF REDUCTIONS FOR SURVEILLANCE TESTING. ON 12/27/79, THE REACTOR SCRAMMED DUE TO A TURBINE TRIP ON HIGH VIBRATION.

LEGEND

[1] Type:
- F-Forced
- S-Scheduled

[2] Reason:
- A-Equipment Failure
- B-Maintenance or Test
- C-Refueling
- D-Regulatory Restriction
- E-Operator Training & License Examination
- F-Administrative
- G-Operational Error
- H-Other

[3] Method:
- 1-Manual
- 2-Manual Scram
- 3-Automatic Scram
- 4-Continuations
- 5-Load Reduction
- 9-Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

FORT CALHOUN

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Nebraska/Washington 19 miles N of Omaha
2.	Electric Reliability Council	MARCA
3.	Type of Reactor	PWR
4.	Power Level (MWT)	1420
5.	Design Electrical Rating (MWE Net)	457
6.	Date of Initial Criticality	August 6, 1973
7.	Date of Commercial Operation	June 20, 1974
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Missouri River

Utility & Contractor Information

10.	Licensee	W. C. Jones Division Manager – Production Operations Omaha Public Power District
11.	Corporate Address	1623 Harney Street Omaha, Nebraska 68102
12.	Corporate Contact	T. E. Short Assistant General Manager
13.	Architect/Engineer	Gibbs & Hill, Inc.
14.	Nuclear Steam Supply System	Combustion Engineering
15.	Constructor	Gibbs & Hill, Durham & Richardson, Inc.

Regulatory Information

16.	IE Region responsible	IV
17.	IE Branch Chief	G. L. Madsen
18.	IE Senior Inspector	T. F. Westerman
19.	IE Principal Inspector	D. Kelley
20.	Licensing Project Manager	P. B. Erickson
21.	Docket Number	50-285
22.	License Number/Date of Issue	DPR-40, 5/24/73
23.	Public Document Room	W. Dale Clark Library 215 S. 15th Street Omaha, Nebraska 68102

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

(1) Concrete shield wall at equipment hatch not installed per FSAR figure 11.2.2 (76-16)

FACILITY ITEMS (PLANS AND PROCEDURES):

MANAGERIAL ITEMS:

(1) Safety Audit and Review Committee review of Facility License Changes. (76-16)

PLANT STATUS:

No significant information

LAST IE SITE INSPECTION DATE: December 10 - 13, 1979 +

INSPECTION REPORT NO: 50-285/79-20 +

SUMMARY

INSPECTION SUMMARY

+ Inspection conducted on November 26, 1979 (79-19): Special inspection of OPPD activities on IE Bulletin 79-14. The inspection involved ten (10) hours on site be one NRC inspector and one other NRC accompanying personnel. No items of noncompliance or deviations or unresolved items identified.

Inspection on December 10-13, 1979 (79-20): Routine, unannounced inspection of the licensee's (1) Procurement Program; (2) Training Activities; (3) Requalification Training Activities; and (4) Preparation for Refueling Activities. The inspection consisted of fifty-four (54) inspector hours on site by two (2) NRC inspectors. No items of noncompliance or deviations were identified.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-19 03L-0	10/5/79	10/30/79		CCW Hx AC-1A inlet valve failure to operate

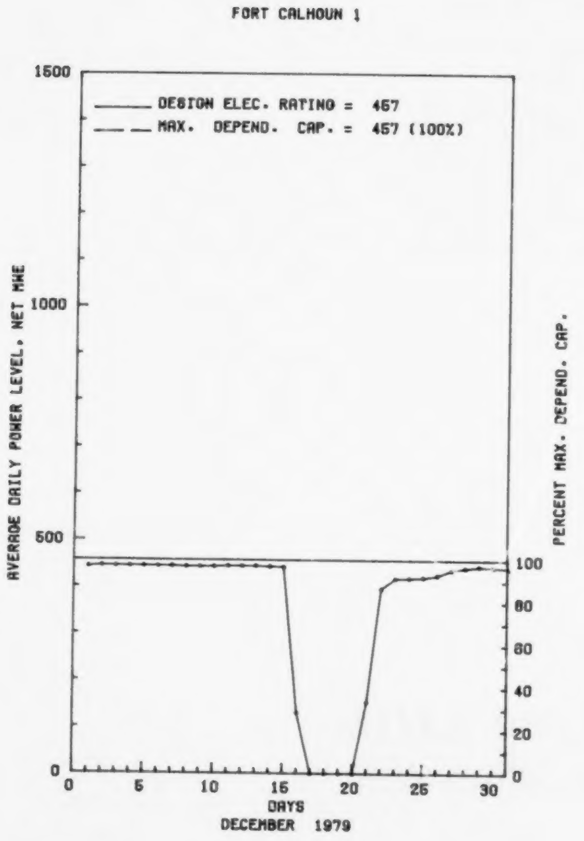
REVIEWED BY OMPA LOEB: Mk Beebe DATE: 02/11/80

REVIEWED BY NRR DOR: P B Erickson DATE: 2/12/80

GPO 871-667

FORT CALHOUN

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

ITEM 21 CALCULATED BY USING A WEIGHTED AVERAGE

1. Docket Number: 50-285
2. Utility Contact: B. J. Hickle (402) 536-4413
3. Licensed Thermal Power (MWt): 1420
4. Nameplate Rating (Gross MWe): 591 x 0.85 = 502
5. Design Electrical Rating (Net MWe): 457
6. Maximum Dependable Capacity (Gross MWe): 481
7. Maximum Dependable Capacity (Net MWe): 457
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	8,760.0	54,937.0
12. Hours Reactor Was Critical	633.8	8,436.6	44,401.6
13. Reactor Reserve Shutdown Hours	.0	173.5	1,309.5
14. Hours Generator On-Line	628.3	8,385.0	43,451.1
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	856,079	11,586,604	53,235,027
17. Gross Electrical Energy (MWH)	283,702	3,855,968	17,660,230
18. Net Electrical Energy (MWH)	269,575	3,666,112	16,687,517
19. Unit Service Factor	84.4	95.7	79.1
20. Unit Availability Factor	84.4	95.7	79.1
21. Unit Capacity Factor (MDC Net)	79.3	91.6	67.0
22. Unit Capacity Factor (DER Net)	79.3	91.6	66.5
23. Unit Forced Outage Rate	15.6	2.3	4.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUELING OUTAGE, JANUARY 18, 8 WEEK DURATION.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
97-06	12/16/79	F	115.7	A	1				REACTOR SHUTDOWN TO CONDUCT RCP SEAL REPLACEMENT.

SUMMARY

OPERATION WAS ROUTINE DURING THE REPORT PERIOD, EXCEPT FOR AN OUTAGE FROM 12/16/79 THROUGH 12/19/79 FOR REACTOR COOLANT PUMP SEAL REPLACEMENT.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B- Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

FORT ST. VRAIN

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Colorado/Weld 35 miles N of Denver (near Platteville)
2.	Electric Reliability Council	WSCC
3.	Type of Reactor	HTGR
4.	Power Level (MWT)	842
5.	Design Electrical Rating (MWE Net)	330
6.	Date of Initial Criticality	January 31, 1974
7.	Date of Commercial Operation	July 1, 1979
8.	Condenser Cooling Method	Towers
9.	Condenser Cooling Water Source	South Platte River/Shallow Wells

Utility & Contractor Information

10.	Licensee	Public Service Company of Colorado
11.	Corporate Address	P. O. Box 840 Denver, Colorado 80201
12.	Corporate Contact	C. K. Millen Senior Vice President
13.	Architect/Engineer	Sargent & Lundy
14.	Nuclear Steam Supply System	General Atomic Company
15.	Constructor	General Atomic Company

Regulatory Information

16.	IE Region responsible	IV
17.	IE Branch Chief	G. L. Madsen
18.	IE Senior Inspector	T. F. Westerman
19.	IE Resident Inspector	M. W. Dickerson
20.	Licensing Project Manager	G. Kuzmycz
21.	Docket Number	50-267
22.	License Number/Date of Issue	DPR-34, 12/21/73
23.	Public Document Room	Greeley Public Library City Complex Building Greeley, Colorado 80631

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

Areas related to potential equipment modification, which are under review and discussion with the NRC include: helium circulator auxiliaries; accident analysis relating to outlet temperature dispersion and orifice settings; hangers for 2" and smaller piping; and plant oscillations.

FACILITY ITEMS (PLANS AND PROCEDURES):

Implementation of 10 CFR 73.55

MANAGERIAL ITEMS:

NONE

PLANT STATUS:

+ Plant shut down for installation of Helium Circulators and maintenance items.

LAST IE SITE INSPECTION DATE: December 3 - 7, 1979 +

INSPECTION REPORT NO: 50-267/79-20

SUMMARY

INSPECTION SUMMARY

+ Inspection conducted during the period October 29 through November 30, 1979 (79-17): Routine, announced inspection of review of plant operations; audits; physical protection; follow-up on bulletins/circulars; calibrations; and review of event reports. No items of noncompliance or deviations were identified.

Information Meeting with Local Public Officials on October 31, 1979 (79-18): Public officials of Greeley and Platteville, Colorado were apprised of the mission of the NRC. Key NRC personnel, including the Resident Inspector, were introduced. Discussion with the local officials included lines of communication with the NRC, the status of the Fort St. Vrain site and community concerns. The meeting involved 11 hours of NRC participation.

Inspection on November 14 - 16, 1979 and November 28-30, 1979 (79-19): Routine, announced inspection of licensee actions taken in response to IE Bulletin 79-02, Revision 2. The inspection involved thirty-five (35) inspection hours on site by one (1) NRC inspector. No items of noncompliance or deviations were identified in the areas inspected.

Inspection on December 3 - 7, 1979 (79-20): Inspection included a review of Security Plan, the Security Program Audit, Security Organization, Protected Area Physical Barriers, and Lighting; Access Control (Identification, Authorization, and Badging): Access Control (Search) Alarm Stations; Testing and Maintenance; Compensatory Measures; Records and Reports; Follow-up on previous items of noncompliance, deviations, and follow-up on IE Circular 79-16. The inspection involved (38) inspector hours on site by one (1) NRC inspector. The licensee was found to be in compliance with NRC requirements within all the areas examined during this inspection.

NRC POWER RESTRICTIONS: Restricted to 70% of Licensed Thermal Power.

REPORTS RECEIVED FROM LICENSEE

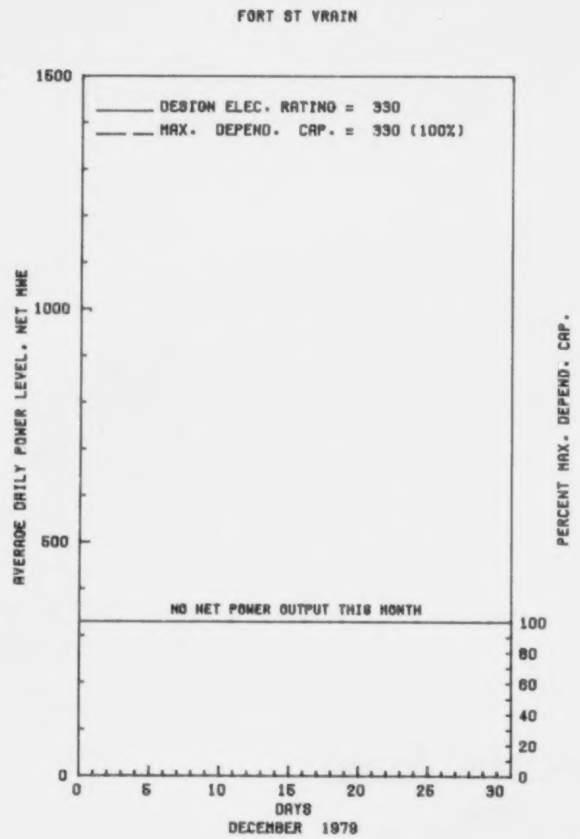
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
79-51 03L-0	10/17/79	11/16/79	Loop 1 surge tank low level which tripped bearing water pumps causing He circulators to be inoperable
79-52 03L-0	10/22/79	11/20/79	"B" instrument air compressor insufficient output
79-53 03L-0	10/24/79	11/20/79	Total primary oxidants greater than 10 ppm with AVG core putlet temp greater than 1200 deg F
79-54 03-X-1	10/24/79	11/28/79	Standby diesel engine 1B tripped on high cooling water temperature due to incorrect valve lineup
79-55 03L-0	10/26/79	11/26/79	Stuck open check valve which would have prevented operation of helium circulators via emergency condensate lines
79-56 03L-0	10/30/79	11/29/79	During monthly check, 7 of 12 helium circulator seal malfunction pressure differential switches tripped outside limits
79-57 03L-0	10/30/79	11/29/79	One reactor pressure transmitter found out of allowable calibration tolerance
79-58 03L-0	12/06/79	11/07/79	Hanger 10A-H-21208 impaired. Constitutes impairment of circulator auxiliary system-bearing water supply
79-59 03L-0	11/09/79	12/06/79	Boiler feed snubber BFS-412 inoperable due to empty oil reservoir

REVIEWED BY OMPA LOEB: T. C. Cintula DATE: 2•[illegible]1•80

REVIEWED BY NRR DOR: DATE: 2. 12. 1980

FORT ST. VRAIN

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-267
2. Utility Contact: J. W. Gahm (303) 785-2253
3. Licensed Thermal Power (MWt): 842
4. Nameplate Rating (Gross MWe): 403 x 0.85 = 343
5. Design Electrical Rating (Net MWe): 330
6. Maximum Dependable Capacity (Gross MWe): 342
7. Maximum Dependable Capacity (Net MWe): 330
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): 231
10. Reasons for Restrictions, If Any:
NRC RESTRIC. PEND. RESOL. OF TEMP. FLUCTUATIONS.

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	4,417.0	4,417.0
12. Hours Reactor Was Critical	113.0	2,532.8	2,532.8
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	.0	982.2	982.2
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,400	477,955	477,955
17. Gross Electrical Energy (MWH)	0	140,796	140,796
18. Net Electrical Energy (MWH)	0	123,584	123,584
19. Unit Service Factor	.0	22.2	22.2
20. Unit Availability Factor	0	22.2	22.2
21. Unit Capacity Factor (MDC Net)	.0	8.5	8.5
22. Unit Capacity Factor (DER Net)	.0	8.5	8.5
23. Unit Forced Outage Rate	.0	65.1	65.1

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: March 3, 1980 due to helium circulator problems.

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
79-14	10/26/79	S	744.0	B	4				SCHEDULED SHUTDOWN FOR PLANT MAINTENANCE AND INSTALLATION OF REGION CONSTRAINT DEVICES.

SUMMARY

THE UNIT REMAINED SHUTDOWN THROUGHOUT THE REPORT PERIOD FOR PLANT MAINTENANCE AND INSTALLATION OF REGION CONSTRAINT DEVICES.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

GINNA

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	New York/Wayne 15 miles NE of Rochester
2.	Electric Reliability Council	NPCC
3.	Type of Reactor	PWR
4.	Power Level (MWT)	1520
5.	Design Electrical Rating (MWE Net)	470
6.	Date of Initial Criticality	November 8, 1969
7.	Date of Commercial Operation	July 1970
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Lake Ontario

Utility & Contractor Information

10.	Licensee	Rochester Gas & Electric Corporation
11.	Corporate Address	89 East Avenue Rochester, New York
12.	Corporate Contact	Leon D. White, Jr., Vice President Electric and Steam Production
13.	Architect/Engineer	Gilbert
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Senior Inspector	R. Keimig
19.	IE Principal Inspector	R. S. Markowski
20.	Licensing Project Manager	J. J. Shea
21.	Docket Number	50-244
22.	License Number/Date of Issue	DPR-18, 9/19/69
23.	Public Document Room	Rochester Public Library Business and Social Science Division 115 South Avenue Rochester, New York 14604

INSPECTION STATUS

ENFORCEMENT STATUS

10 CFR 50.54 TITLED "CONDITIONS OF LICENSES" STATES IN PART: "...THE FOLLOWING SHALL BE DEEMED CONDITIONS FOR EVERY LICENSE ISSUED:...(E) THE LICENSE SHALL BE SUBJECT TO...AMENDMENT FOR CAUSE...." CONTRARY TO THE ABOVE, ON 8/24-25/79, THE INSPECTORS OBSERVED THAT THE LICENSEE HAS NOT IMPLEMENTED THE ABOVE REFERENCED MODIFICATIONS. (7916 INFRACTION)

TS 6.8.1 STATES IN PART, THAT: "WRITTEN PROCEDURES SHALL BE ESTABLISHED, IMPLEMENTED & MAINTAINED COVERING... FIRE PROTECTION PROGRAM IMPLEMENTATION". CONTRARY TO THE ABOVE, ON 10/25/79, A WELDING OPERATION AUTHORIZED BY WELDING PERMIT #8002 WAS OBSERVED TO BE CONDUCTED WITHIN 5 FT OF COMBUSTIBLE MATERIALS, WITHOUT THE BENEFIT OF FLAMEPROOF COVERS, & WITHIN 20 FT OF AN UNCOVERED MOVEABLE DRUM OF OIL. (7916 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

+ The plant returned to operation on 12/16 following repair of the B steam generator tube leak.

+ Evaluations into the causes and repair of a linear indication on the pressurizer power operated relief valve yokes are continuing. Licensee NDE examination of replacement yokes received onsite revealed cracks in the new cast steel yokes. The NRC staff is evaluating the latest information for consideration of further action.

FACILITY ITEMS (PLANS AND PROCEDURES):

+ Plant shutdown for the annual refueling and maintenance outage is scheduled for March 15, 1980.

MANAGERIAL ITEMS:

None.

PLANT STATUS:

+ The plant is operating at full rated power.

+ LAST IE SITE INSPECTION DATE: 1/17/80

+ INSPECTION REPORT NO: 80-01

SUMMARY

INSPECTION SUMMARY

+ Jun 26-29, Aug 28-31 & Oct 10-12 (79-10): Investigation conducted by three region based investigators to review concrete test data, structural integrity test reports, tendon surveillance records and quality assurance records. The containment exterior surfaces were examined and a tendon lift off test was witnessed. The investigation involved 131 investigator-hours at the licensee's corporate office and site. No items of noncompliance were identified.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

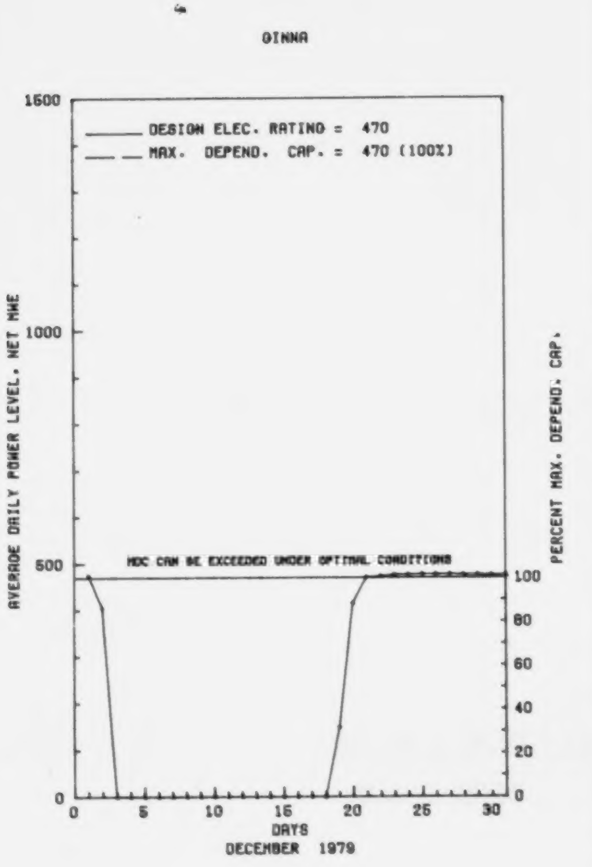
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
NONE				

REVIEWED BY OMPA LOEB: T. C. Cintula DATE: 2-11-80

REVIEWED BY NRR DOR: ______ DATE: ______

GINNA

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

CAPACITY FACTORS CALCULATED WITH A WEIGHTED AVERAGE

1. Docket Number: 50-244
2. Utility Contact: A. E. McNamara (716) 546-2700 x291-205
3. Licensed Thermal Power (MWt): 1520
4. Nameplate Rating (Gross MWe): 608 x 0.85 = 517
5. Design Electrical Rating (Net MWe): 470
6. Maximum Dependable Capacity (Gross MWe): 490
7. Maximum Dependable Capacity (Net MWe): 470
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	8,760.0	88,512.0
12. Hours Reactor Was Critical	339.7	6,477.3	67,679.8
13. Reactor Reserve Shutdown Hours	42.5	75.1	1,621.8
14. Hours Generator On-Line	327.5	6,378.0	65,956.8
15. Unit Reserve Shutdown Hours	.0	.0	8.5
16. Gross Thermal Energy (MWH)	486,144	9,345,888	88,275,802
17. Gross Electrical Energy (MWH)	159,810	3,112,551	28,685,791
18. Net Electrical Energy (MWH)	152,082	2,960,510	27,161,548
19. Unit Service Factor	44.0	72.8	74.5
20. Unit Availability Factor	44.0	72.8	74.5
21. Unit Capacity Factor (MDC Net)	43.5	71.9	65.3
22. Unit Capacity Factor (DER Net)	43.5	71.9	67.7
23. Unit Forced Outage Rate	56.0	6.3	9.9

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUELING & MAINTENANCE - MARCH 15, 1980.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
6	12/02/79	F	416.5	A	1	79-22	HB	HTEXCH	"B" STEAM GENERATOR TUBE LEAK - PRESSURIZER RELIEF LINE NOZZLE REPAIR - BORIC ACID TANKS, LOW BORON CONCENTRATION.

SUMMARY

AN OUTAGE OCCURRED FROM 12/02/79 THROUGH 12/18/79 FOR REPAIR OF A TUBE LEAK IN THE "B" STEAM GENERATOR. OPERATION WAS AT NEAR FULL POWER THE REMAINDER OF THE REPORT PERIOD.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

HADDAM NECK

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Connecticut/Middlesex 13 miles E of Meriden
2.	Electric Reliability Council	NPCC
3.	Type of Reactor	PWR
4.	Power Level (MWT)	1825
5.	Design Electrical Rating (MWE Net)	575
6.	Date of Initial Criticality	July 24, 1967
7.	Date of Commercial Operation	January 1, 1968
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Connecticut River

Utility & Contractor Information

10.	Licensee	Connecticut Yankee Atomic Power Co.
11.	Corporate Address	P. O. Box 270 Hartford, Connecticut
12.	Corporate Contact	D. Switzer, President
13.	Architect/Engineer	Stone & Webster
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Senior Inspector	R. Keimig
19.	IE Principal Inspector	W. J. Lazarus
20.	Licensing Project Manager	R. D. Silver
21.	Docket Number	50-213
22.	License Number/Date of Issue	DPR-61, 12/27/74
23.	Public Document Room	Russell Library 119 Broad Street Middletown, Connecticut 06457

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

None

FACILITY ITEMS (PLANS AND PROCEDURES):

None.

MANAGERIAL ITEMS:

PLANT STATUS:

Nothing significant to report.

♦ LAST IE SITE INSPECTION DATE: 12/17-21/79

INSPECTION REPORT NO: 79-21

SUMMARY

INSPECTION SUMMARY

♦ Nov 7-8 (79-19): Special, unannounced inspection of the licensee's response to Bulletin 79-19, including: regulatory requirements, procedures, training, audits, records of shipments and onsite observations. The inspection involved 12 hours on site by one NRC region-based inspector. One item of noncompliance was identified (failure to comply with requirements of 10 CFR 71).

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-005E	11/16/79	11/26/79	Environmental	Sodium Hypochlorite spill

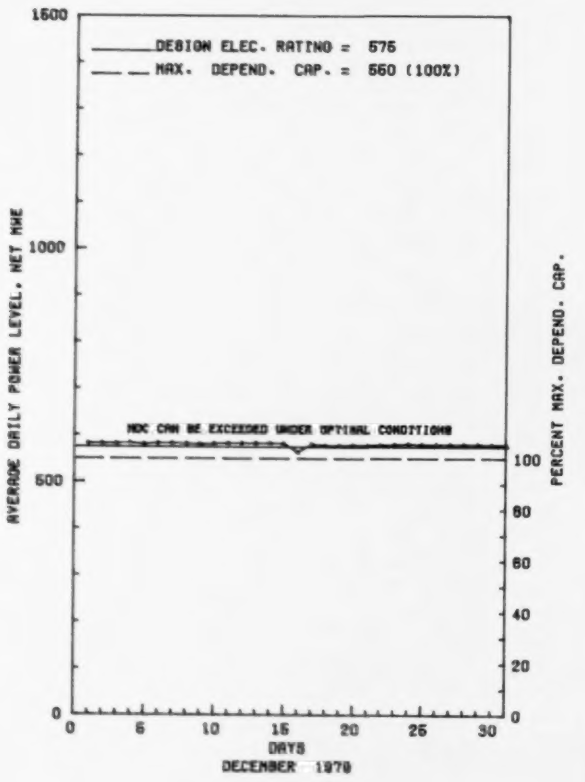
REVIEWED BY OMPA LOEB: T. C. Cintula DATE: 2·11·80

REVIEWED BY NRR DOR: DATE: 2/13/80

HADDAM NECK

AVERAGE DAILY POWER LEVEL (MWe)

HADDAM NECK



OPERATING STATUS

CAPACITY FACTORS
CALCULATED WITH A
WEIGHTED AVERAGE

1. Docket Number: 50-213
2. Utility Contact: C. Kokoszka (203) 267-2556
3. Licensed Thermal Power (MWt): 1825
4. Nameplate Rating (Gross MWe): 667 x 0.9 = 600
5. Design Electrical Rating (Net MWe): 575
6. Maximum Dependable Capacity (Gross MWe): 577
7. Maximum Dependable Capacity (Net MWe): 550
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	8,760.0	105,192.0
12. Hours Reactor Was Critical	744.0	7,541.2	91,196.7
13. Reactor Reserve Shutdown Hours	.0	254.5	1,152.6
14. Hours Generator On-Line	744.0	7,427.6	86,940.2
15. Unit Reserve Shutdown Hours	.0	233.5	369.9
16. Gross Thermal Energy (MWH)	1,356,111	13,202,066	150,141,149
17. Gross Electrical Energy (MWH)	450,790	4,322,052	49,354,466
18. Net Electrical Energy (MWH)	430,745	4,116,339	46,954,763
19. Unit Service Factor	100.0	84.8	82.6
20. Unit Availability Factor	100.0	87.5	83.0
21. Unit Capacity Factor (MDC Net)	105.3	85.4	82.6
22. Unit Capacity Factor (DER Net)	100.7	81.7	76.8
23. Unit Forced Outage Rate	.0	.4	7.3

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUELING, MAY 1980, APPROX. 8 TO 12 WEEKS.

25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	NONE								

SUMMARY

OPERATION WAS AT NEAR FULL POWER DURING THE REPORT PERIOD, WITH A BRIEF LOAD REDUCTION ON 12/16/79 FOR TURBINE STOP VALVE TRIP TESTING.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

HATCH 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Georgia/Appling 11 miles N of Baxley
2.	Electric Reliability Council	Member of SE Electric Reliability Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2436
5.	Design Electrical Rating (MWE Net)	786
6.	Date of Initial Criticality	September 12, 1974
7.	Date of Commercial Operation	December 31, 1975
8.	Condenser Cooling Method	Once Through With Forced Draft Cooling Towers
9.	Condenser Cooling Water Source	Altamaha River

Utility & Contractor Information

10.	Licensee	Georgia Power Company
11.	Corporate Address	270 Peachtree Street, NE Atlanta, Georgia 30303
12.	Corporate Contact	J. H. Miller Executive Vice President
13.	Architect/Engineer	Southern Services, Inc.
14.	Nuclear Steam Supply System	GE
15.	Constructor	Georgia Power Company

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. Lewis
18.	IE Senior Inspector	H. C. Dance
19.	IE Resident Inspector	R. F. Rogers
20.	Licensing Project Manager	J. N. Hannon
21.	Docket Number	50-321
22.	License Number/Date of Issue	DPR-57, 8/6/74
23.	Public Document Room	Appling County Public Library Parker Street Baxley, Georgia 31513

INSPECTION STATUS

ENFORCEMENT STATUS

AS REQUIRED BY SECTION 5.6 OF THE ENVIRONMENTAL TECHNICAL SPECIFICATIONS, DETAILED WRITTEN PROCEDURES SHALL BE PREPARED AND FOLLOWED FOR ALL ACTIVITIES INVOLVED IN IMPLEMENTING THE ETS. CONTRARY TO THE ABOVE, THE TEMPERATURE OF THE WEATHER STATION BUILDING WAS GREATER THAN 75 DEGREE F AND THE AIR CONDITIONER WAS NOT FUNCTIONING PROPERLY AS REQUIRED BY PROCEDURE HNP-7900, PARAGRAPH 4.A. IN ADDITION, CONTRARY TO PARAGRAPH 4.J OF THE SAME PROCEDURE, IT WAS OBSERVED THAT DIFFERENCE OF MORE THAN 20 DEGREES EXISTED BETWEEN THE TOP AND BOTTOM WIND DIRECTION INDICATORS AND THE REQUIRED ADDITONAL CHECKS WERE NOT PERFORMED.
(7930 DEFICIENCY)

AS REQUIRED BY TECHNICAL SPECIFICATION 6.8.1, WRITTEN PROCEDURES SHALL BE ESTABLISHED AND IMPLEMENTED FOR SAFETY RELATED ACTIVITIES. HATCH NUCLEAR PLANT PROCEDURE NO. 314, KEY CONTROL, PARAGRAPH B.3, STATES THAT KEYS FOR KEY OPERATED SWITCHES ON THE CONTROL BOARD ARE TO BE KEPT ON THE KEY BOARD WHEN NOT IN USE, AND SPECIFICALLY STATES KEYS ARE NOT TO BE LEFT IN THE SWITCHES. HATCH NUCLEAR PLANT PROCEDURE NO. 17, RELIEF OF PERSONNEL, PARAGRAPH D.1 STATES THAT THE ONCOMING PLANT OPERATOR IS TO REVIEW ENTRIES MADE IN THE OPERATOR'S LOG AND INITIAL EACH PAGE. IT FURTHER REQUIRES THAT ENTRIES MADE THE PREVIOUS TWO SHIFTS SHOULD BE REVIEWED PRIOR TO ASSUMING SHIFT RESPONSIBILITIES. CONTRARY TO THE ABOVE: 1. ON OCTOBER 4, 1979, THREE KEYS ASSOCIATED WITH THE HIGH PRESSURE COOLANT INJECTION SYSTEM AND TWO KEYS ASSOCIATED WITH THE REACTOR CORE ISOLATION COOLING SYSTEM WERE BEING MAINTAINED IN THEIR RESPECTIVE SWITCHES ON THE CONTROL PANEL ON UNIT 2. 2. THE UNIT 1 PLANT OPERATOR ON OCTOBER 4, 1979 AND THE UNIT 2 PLANT OPERATOR ON OCTOBER 3, 1979, DID NOT INITIAL THE PREVIOUS TWO SHIFT ENTRIES PRIOR TO ASSUMING DAY SHIFT RESPONSIBILITIES. A SIMILAR ITEM OF THE FIRST EXAMPLE WAS BROUGHT TO YOUR ATTENTION IN OUR LETTER DATED AUGUST 3, 1979.
(7932 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

PLANT OPERATING AT FULL POWER.

* LAST IE SITE INSPECTION DATE: NOVEMBER 10 - DECEMBER 7, 1979

* INSPECTION REPORT NO: 50-321/79-37

SUMMARY

INSPECTION SUMMARY

* INSPECTION OCTOBER 16-19 AND 23 (79-30): THIS ROUTINE UNANNOUNCED INSPECTION INVOLVED 14 INSPECTOR-HOURS ON SITE IN THE AREAS OF RADIOLOGICAL ENVIRONMENTAL MONITORING INCLUDING MANAGEMENT CONTROLS, QUALITY CONTROL OF ANALYTICAL MEASUREMENTS, INSPECTION OF ENVIRONMENTAL MONITORING STATIONS AND SAMPLE COLLECTION, REVIEW OF ENVIRONMENTAL MONITORING DATA, REVIEW OF ENVIRONMENTAL PROCEDURES AND THEIR IMPLEMENTATION. OF THE FIVE AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN FOUR AREAS, ONE APPARENT ITEM OF NONCOMPLIANCE WAS FOUND IN ONE AREA (DEFICIENCY - FAILURE TO FOLLOW PROCEDURES IN PERFORMING DAILY OBSERVATION CHECKS AT THE METEOROLOGICAL STATION).

INSPECTION OCTOBER 6 - NOVEMBER 9 (79-35): THIS INSPECTION INVOLVED 45 INSPECTOR-HOURS ON SITE OF TECHNICAL SPECIFICATION COMPLIANCE, REPORTABLE OCCURRENCES, HOUSEKEEPING, OPERATOR PERFORMANCE, OVERALL PLANT OPERATIONS, QUALITY ASSURANCE PRACTICES, STATION AND CORPORATE MANAGEMENT PRACTICES, CORRECTIVE AND PREVENTATIVE MAINTENANCE ACTIVITIES, SITE SECURITY PROCEDURES, RADIATION CONTROL ACTIVITIES, AND SURVEILLANCE ACTIVITIES. OF THE ELEVEN AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION NOVEMBER 26-28 (79-36): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 11 INSPECTOR-HOURS ON SITE IN THE AREAS OF ROUTINE MAINTENANCE AND PLANT TOUR. OF THE TWO AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

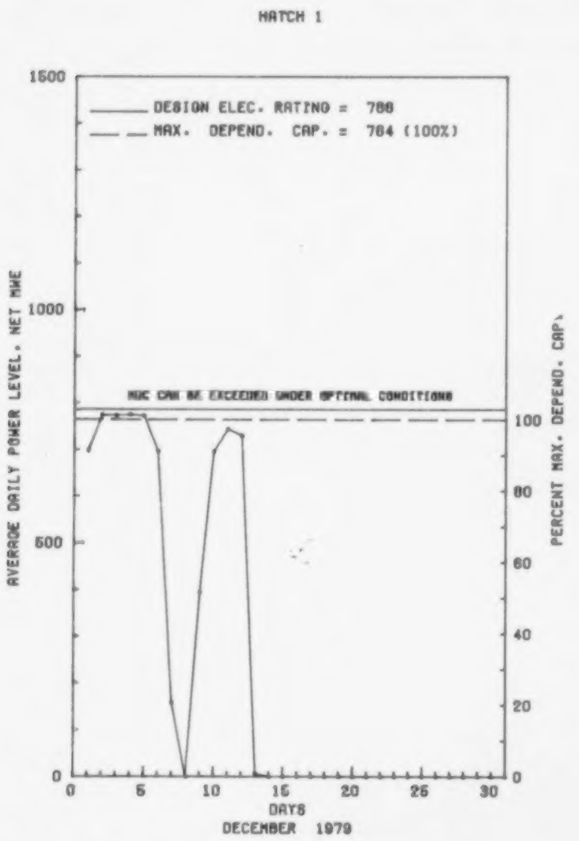
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-095/ 03L-0	11/26/79	12/14/79	30 DAY LER	SURVEILLANCE OF BATTERY INDIVIDUAL CELL PERFORMED LATE.
79-096/ 03L-0	11/27/79	12/11/79	30 DAY LER	OXYGEN ANALYZER OUT OF TOLERANCE.

REVIEWED BY OMPA LOEB: John J. Briggs DATE: 2-11-80

REVIEWED BY NRR DOR: John N. Hannon DATE: 2-13-80

HATCH 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-321
2. Utility Contact: G. L. Johnson (912) 367-7781
3. Licensed Thermal Power (MWt): 2436
4. Nameplate Rating (Gross MWe): 1000 x 0.85 = 850
5. Design Electrical Rating (Net MWe): 786
6. Maximum Dependable Capacity (Gross MWe): 797
7. Maximum Dependable Capacity (Net MWe): 764
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	8,760.0	35,064.0
12. Hours Reactor Was Critical	321.6	5,090.1	25,778.5
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	251.9	4,784.0	24,268.4
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	556,831	10,744,778	50,353,055
17. Gross Electrical Energy (MWH)	181,910	3,515,430	16,270,940
18. Net Electrical Energy (MWH)	170,014	3,337,875	15,461,933
19. Unit Service Factor	33.9	54.6	69.2
20. Unit Availability Factor	33.9	54.6	69.2
21. Unit Capacity Factor (MDC Net)	29.9	49.9	57.7
22. Unit Capacity Factor (DER Net)	29.1	48.5	56.1
23. Unit Forced Outage Rate	66.1	14.1	16.1

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 01/07/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
79-20	12/07/79	F	21.5	H	3		CH	INSTRU	RX SCRAM - LOW WATER LEVEL SIGNAL DUE TO REMOVING 'A' RFPT MGU CONTROLLER.
79-21	12/08/79	F	17.7	A	1		MB	ELECON	SHUTDOWN DUE TO LOSS OF MOTOR CONTROL CENTER R24-5035.
79-22	12/09/79	F	.0	H	5		RA	INSTRU	REDUCTION DUE TO PROBLEMS WITH TIP MACHINE AND PCIOMR.
79-23	12/13/79	F	452.9	A	1		HA	GENERA	SHUTDOWN DUE TO GROUND FAULT IN THE MAIN GENERATOR ROTOR.

SUMMARY

THE UNIT EXPERIENCED A REACTOR SCRAM, A SHUTDOWN, AND A LOAD REDUCTION BEFORE BEING TAKEN OFF-LINE FROM 12/13/79 THROUGH THE END OF THE REPORT PERIOD DUE TO A GROUND FAULT IN THE MAIN GENERATOR ROTOR.

LEGEND

[1] Type:
- F–Forced
- S–Scheduled

[2] Reason:
- A–Equipment Failure
- B–Maintenance or Test
- C–Refueling
- D–Regulatory Restriction
- E–Operator Training & License Examination
- F–Administrative
- G–Operational Error
- H–Other

[3] Method:
- 1–Manual
- 2–Manual Scram
- 3–Automatic Scram
- 4–Continuations
- 5–Load Reduction
- 9–Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

HATCH 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Georgia/Appling 11 miles N of Baxley
2.	Electric Reliability Council	Member of SE Electric Reliability Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2436
5.	Design Electrical Rating (MWE Net)	784
6.	Date of Initial Criticality	July 4, 1978
7.	Date of Commercial Operation	September 5, 1979
8.	Condenser Cooling Method	Cooling Towers
9.	Condenser Cooling Water Source	Altamaha River

Utility & Contractor Information

10.	Licensee	Georgia Power Company
11.	Corporate Address	270 Peach Street, NE Atlanta, Georgia 30303
12.	Corporate Contact	J. H. Miller Executive Vice President
13.	Architect/Engineer	Southern Services, Inc./Bechtel
14.	Nuclear Steam Supply System	GE
15.	Constructor	Georgia Power Company

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. Lewis
18.	IE Senior Inspector	H. C. Dance
19.	IE Resident Inspector	R. F. Rogers
20.	Licensing Project Manager	J. N. Hannon
21.	Docket Number	50-366
22.	License Number/Date of Issue	NPF-5/June 13, 1978
23.	Public Document Room	Appling County Public Library Parker Street Baxley, Georgia 31513

INSPECTION STATUS

ENFORCEMENT STATUS

AS REQUIRED BY SECTION 5.6 OF THE ENVIRONMENTAL TECHNICAL SPECIFICATIONS, DETAILED WRITTEN PROCEDURES SHALL BE PREPARED AND FOLLOWED FOR ALL ACTIVITIES INVOLVED IN IMPLEMENTING THE ETS. CONTRARY TO THE ABOVE, THE TEMPERATURE OF THE WEATHER STATION BUILDING WAS GREATER THAN 75 DEGREE F AND THE AIR CONDITIONER WAS NOT FUNCTIONING PROPERLY AS REQUIRED BY PROCEDURE HNP-7980, PARAGRAPH 4.A. IN ADDITION, CONTRARY TO PARAGRAPH 4.J OF THE SAME PROCEDURE, IT WAS OBSERVED THAT DIFFERENCE OF MORE THAN 20 DEGREES EXISTED BETWEEN THE TOP AND BOTTOM WIND DIRECTION INDICATORS AND THE REQUIRED ADDITONAL CHECKS WERE NOT PERFORMED.
(7934 DEFICIENCY)

AS REQUIRED BY TECHNICAL SPECIFICATION 6.8.1, WRITTEN PROCEDURES SHALL BE ESTABLISHED AND IMPLEMENTED FOR SAFETY RELATED ACTIVITIES. HATCH NUCLEAR PLANT PROCEDURE NO. 314, KEY CONTROL, PARAGRAPH B.3, STATES THAT KEYS FOR KEY OPERATED SWITCHES ON THE CONTROL BOARD ARE TO BE KEPT ON THE KEY BOARD WHEN NOT IN USE, AND SPECIFICALLY STATES KEYS ARE NOT TO BE LEFT IN THE SWITCHES. HATCH NUCLEAR PLANT PROCEDURE NO. 17, RELIEF OF PERSONNEL, PARAGRAPH D.1 STATES THAT THE ONCOMING PLANT OPERATOR IS TO REVIEW ENTRIES MADE IN THE OPERATOR'S LOG AND INITIAL EACH PAGE. IT FURTHER REQUIRES THAT ENTRIES MADE THE PREVIOUS TWO SHIFTS SHOULD BE REVIEWED PRIOR TO ASSUMING SHIFT RESPONSIBILITIES. CONTRARY TO THE ABOVE: 1. ON OCTOBER 4, 1979, THREE KEYS ASSOCIATED WITH THE HIGH PRESSURE COOLANT INJECTION SYSTEM AND TWO KEYS ASSOCIATED WITH THE REACTOR CORE ISOLATION COOLING SYSTEM WERE BEING MAINTAINED IN THEIR RESPECTIVE SWITCHES ON THE CONTROL PANEL ON UNIT 2. 2. THE UNIT 1 PLANT OPERATOR ON OCTOBER 4, 1979 AND THE UNIT 2 PLANT OPERATOR ON OCTOBER 3, 1979, DID NOT INITIAL THE PREVIOUS TWO SHIFT ENTRIES PRIOR TO ASSUMING DAY SHIFT RESPONSIBILITIES. A SIMILAR ITEM OF THE FIRST EXAMPLE WAS BROUGHT TO YOUR ATTENTION IN OUR LETTER DATED AUGUST 3, 1979.
(7936 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

PLANT OPERATING AT FULL POWER.

+ LAST IE SITE INSPECTION DATE: NOVEMBER 10 - DECEMBER 7, 1979

+ INSPECTION REPORT NO: 50-366/79-41

SUMMARY

INSPECTION SUMMARY

+ INSPECTION OCTOBER 16-19 AND 23 (79-34): THIS ROUTINE UNANNOUNCED INSPECTION INVOLVED 14 INSPECTOR-HOURS ON SITE IN THE AREAS OF RADIOLOGICAL ENVIRONMENTAL MONITORING INCLUDING MANAGEMENT CONTROLS, QUALITY CONTROL OF ANALYTICAL MEASUREMENTS, INSPECTION OF ENVIRONMENTAL MONITORING STATIONS AND SAMPLE COLLECTION, REVIEW OF ENVIRONMENTAL MONITORING DATA, REVIEW OF ENVIRONMENTAL PROCEDURES AND THEIR IMPLEMENTATION. OF THE FIVE AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN FOUR AREAS. ONE APPARENT ITEM OF NONCOMPLIANCE WAS FOUND IN ONE AREA (DEFICIENCY - FAILURE TO FOLLOW PROCEDURES IN PERFORMING DAILY OBSERVATION CHECKS AT THE METEOROLOGICAL STATION).

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-118/ 03L-0	11/03/79	11/15/79	30 DAY LER	LOW PRESSURE COOLANT INJECTION VALVE INOPERABLE.
79-119/ 03L-0	11/13/79	11/28/79	30 DAY LER	APRM INOPERABLE SETPOINT DRIFT.
79-125/ 03L-0	11/14/79	12/12/79	30 DAY LER	WASTE TANK SAMPLE RECIRCULATION INADEQUATE.
79-126/ 03L-0	11/21/79	12/14/79	30 DAY LER	PLANT SERVICE WATER PUMP INOPERABLE.
79-129/ 03L-0	11/29/79	12/14/79	30 DAY LER	HIGH PRESSURE COOLANT INJECTION AUXILIARY OIL PUMP INOPERABLE.

INSPECTION OCTOBER 6 - NOVEMBER 9 (79-39): THIS INSPECTION INVOLVED 45 INSPECTOR-HOURS ON SITE OF TECHNICAL SPECIFICATION COMPLIANCE, REPORTABLE OCCURRENCES, HOUSEKEEPING, OPERATOR PERFORMANCE, OVERALL PLANT OPERATIONS, QUALITY ASSURANCE PRACTICES, STATION AND CORPORATE MANAGEMENT PRACTICES, CORRECTIVE AND PREVENTATIVE MAINTENANCE ACTIVITIES, SITE SECURITY PROCEDURES, RADIATION CONTROL ACTIVITIES, AND SURVEILLANCE ACTIVITIES. OF THE ELEVEN AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

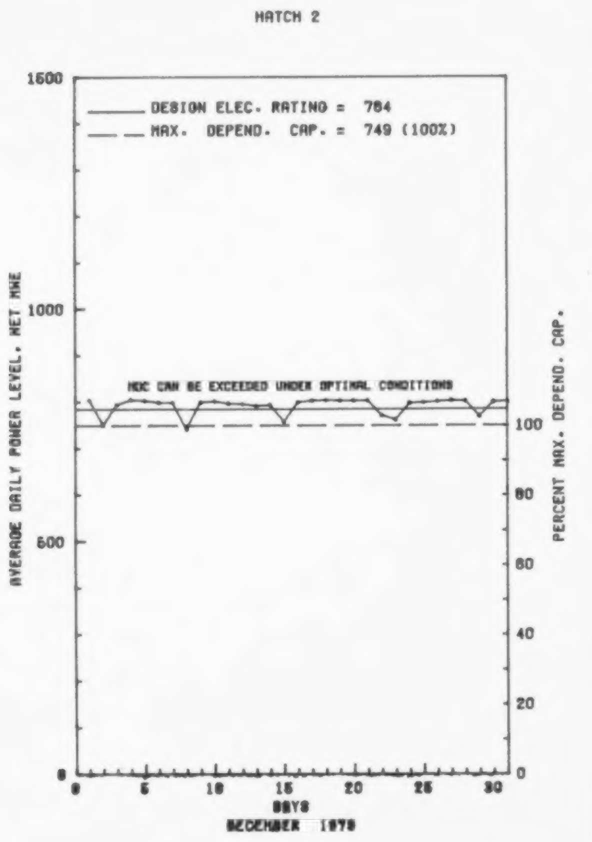
INSPECTION NOVEMBER 26-28 (79-40): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 12 INSPECTOR-HOURS ON SITE IN THE AREAS OF ROUTINE MAINTENANCE AND PLANT TOUR. OF THE TWO AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

REVIEWED BY OMPA LOEB: John J. Riggs **DATE:** 2-11-80

REVIEWED BY NRR DOR: John N. Hannon **DATE:** 2-13-80

HATCH 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-366
2. Utility Contact: G. L. Johnson (912) 367-7781
3. Licensed Thermal Power (MWt): 2636
4. Nameplate Rating (Gross MWe): 809
5. Design Electrical Rating (Net MWe): 784
6. Maximum Dependable Capacity (Gross MWe): 782
7. Maximum Dependable Capacity (Net MWe): 749
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	2,833.0	2,833.0
12. Hours Reactor Was Critical	744.0	2,663.5	2,663.5
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	744.0	2,414.7	2,414.7
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,793,810	5,494,062	5,494,062
17. Gross Electrical Energy (MWH)	612,090	1,835,960	1,835,960
18. Net Electrical Energy (MWH)	588,617	1,757,137	1,757,137
19. Unit Service Factor	100.0	85.2	85.2
20. Unit Availability Factor	100.0	85.2	85.2
21. Unit Capacity Factor (MDC Net)	105.6	82.8	82.8
22. Unit Capacity Factor (DER Net)	100.9	79.1	79.1
23. Unit Forced Outage Rate	.0	7.5	7.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
VENT HEADER DEFLECTOR & SURVEIL - 03/01-04/13.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
NONE									

SUMMARY

OPERATION WAS AT NEAR FULL POWER DURING THE REPORT PERIOD, WITH BRIEF REDUCTIONS FOR MAINTENANCE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

HUMBOLDT BAY

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	California/Humboldt 4 miles S of Eureka
2.	Electric Reliability Council	WSCC
3.	Type of Reactor	BWR
4.	Power Level (MWT)	220
5.	Design Electrical Rating (MWE Net)	65
6.	Date of Initial Criticality	February 16, 1963
7.	Date of Commercial Operation	August 1963
8.	Condenser Cooling Method	Circulating sea water
9.	Condenser Cooling Water Source	Pacific Ocean (Humboldt Bay)

Utility & Contractor Information

10.	Licensee	Pacific Gas & Electric Co.
11.	Corporate Address	77 Beale Street San Francisco, California 94106
12.	Corporate Contact	P. A. Crane, Jr. Asst. General Counsel
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	GE
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	V
17.	IE Branch Chief	J. L. Crews
18.	IE Senior Inspector	D. M. Sternberg
19.	IE Principal Inspector	J. D. Carlson
20.	Licensing Project Manager	V. L. Rooney
21.	Docket Number	50-133
22.	License Number/Date of Issue	DPR-7, 8/28/62
23.	Public Document Room	Humboldt County Library 636 F Street Eureka, California 95501

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST (CITE RO REPORT NUMBER, IF APP.)

SYSTEMS AND COMPONENT PROBLEMS:

None.

FACILITY ITEMS (PLANS AND PROCEDURES):

The plant has been shut down since July 2, 1976.

Several items remaining to be completed during the outage have been deferred pending resolution of the seismic issues at the plant site. A date for return to operations has not been established.

MANAGERIAL ITEMS:

None

PLANT STATUS:

Plant is shutdown.

LAST IE SITE INSPECTION DATE: 12/18-20/79+

INSPECTION REPORT NO: 50-133/79-04+

SUMMARY

INSPECTION SUMMARY

+ Inspection on December 18-20, 1979 (Report No 50-133/79-04) Areas Inspected: Routine, unannounced inspection of operations; design, design changes and modifications; Onsite Review Committee; requalification training; maintenance; followup on LER's, Bulletins and Circulars; and independent inspection. The inspection involved 16 inspector hours onsite by one NRC inspector.

Results: No items of noncompliance or deviations were identified.

LICENSE STATUS

Order issued 5/21/76 requiring Utility to prove seismic qualification before resuming operation of unit after outage of 7/2/76.

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
NONE				

REVIEWED BY OMPA LOEB: John J. Rizzo DATE: 2-11-80

REVIEWED BY NRR DOR: ______ DATE: 2/11/80

GPO 871-667

HUMBOLDT BAY

AVERAGE DAILY POWER LEVEL (MWe)

OPERATING STATUS

1. Docket Number: 50-133
2. Utility Data Prepared By: B. C. Getty (707) 443-2787
3. Licensed Thermal Power (MWt): 220
4. Nameplate Rating (Gross MWe): 77 X 0.85 = 65
5. Design Electrical Rating (Net MWe): 65
6. Maximum Dependable Capacity (Gross MWe): 0
7. Maximum Dependable Capacity (Net MWe): 0
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons

None

9. Power Level To Which Restricted, If Any (Net MWe): None
10. Reasons For Restrictions, If Any

This ... ...ive

11. Hours In Reporting Period
12. Number Of Hours Reactor Was Cri...
13. Reactor Reserve Shutdow...
14. Hours Generator...
15. Unit R...
16. ...H)
17. ...erated (MWH)
18. Net ...gy Generated (MWH)
19. Unit S... ce Factor
20. Unit Availability Factor
21. Unit Capacity Factor (Using MDC Net) 1/
22. Unit Capacity Factor (Using DER Net) 1/
23. Unit Forced Outage Rate
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

None

25. If Shut Down At End Of Report Period, Estimated Date of Startup: UNKNOWN

NO DECISION ON FUTURE OPERATION
SHUTDOWN INDEFINITELY

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
SEE SUMMARY BELOW									

SUMMARY

THE UNIT WAS SHUTDOWN ON 7/2/76 FOR SEISMIC MODIFICATIONS. THE UNIT REMAINS IN THE COLD SHUTDOWN CONDITION.

LEGEND

[1]Type:
- F--Forced
- S--Scheduled

[2]Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3]Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4]Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5]Exhibit H: Same source.

INDIAN POINT 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	New York/Westchester 25 miles N of New York City
2.	Electric Reliability Council	NPCC
3.	Type of Reactor	PWR/fossil superheat
4.	Power Level (MWT)	615
5.	Design Electrical Rating (MWE Net)	265
6.	Date of Initial Criticality	August 2, 1962
7.	Date of Commercial Operation	October 1962
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Hudson River

Utility & Contractor Information

10.	Licensee	Consolidated Edison Company of New York, Inc.
11.	Corporate Address	4 Irving Place New York, New York
12.	Corporate Contact	W. Cahill, Vice President
13.	Architect/Engineer	Utility
14.	Nuclear Steam Supply System	B&W
15.	Constructor	Utility

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Senior Inspector	R. Keimig
19.	IE Resident Inspector	T. Rebelowski (914-739-9360) FTS 656-9220
20.	Licensing Project Manager	L. Olshan x27144
21.	Docket Number	50-03
22.	License Number/Date of Issue	DPR-5, 3/26/62
23.	Public Document Room	White Plains Public Library 100 Martine Avenue White Plains, New York 10601

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

```
    INSPECTION STATUS

    OTHER ITEMS OF INTEREST

    SYSTEMS AND COMPONENT PROBLEMS:

    None

    FACILITY ITEMS (PLANS AND PROCEDURES):

    None

    MANAGERIAL ITEMS:

    None

    PLANT STATUS.

    Defueled.

  LAST IE SITE INSPECTION DATE:  7/24-27/79

         INSPECTION REPORT NO:  79-02

 3.  SUMMARY

    INSPECTION SUMMARY

    None.
```

LICENSE STATUS

The unit produced electricity to the expiration date (October 31, 1974) of delay granted by the AEC for installation of an ECCS system. The ECCS system was not subsequently installed and the unit was defueled on January 30, 1976.

On February 11, 1980, the NRC requested the utility to show cause as to why the operating authority of the license should not be revoked and why the licensee should not submit a plan within 120 days of this order to decommission the unit.

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
NONE				

REVIEWED BY OMPA LOEB: T. C. Cintula DATE: 2·1[illegible]·80

REVIEWED BY NRR DOR: DATE: 2 13 80

GPO 871-667

INDIAN POINT 1

AVERAGE DAILY POWER LEVEL (MWe)

OPERATING STATUS

1. Docket Number: 50-003
2. Utility Data Prepared By: L. J. Kawula (914)-694-6000 x209
3. Licensed Thermal Power (MWt): 615
4. Nameplate Rating (Gross MWe): 275
5. Design Electrical Rating (Net MWe): 265
6. Maximum Dependable Capacity (Gross MWe): 0
7. Maximum Dependable Capacity (Net MWe): 0
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons
None
9. Power Level To Which Restricted, If Any (Net MWe): None
10. Reasons For Restrictions, If Any:

	This Month	Yr-to-Date
11. Hours In Reporting Period		
12. Number Of Hours Reactor Was Critical		
13. Reactor Reserve Shutdown Hours		
14. Hours Generator On-Line		
15. Unit Reserve Shutdown Ho		
16. Gross Thermal E		
17. Gros		
20. Unit A		
21. Unit Capacity Factor (U Net) 1/		
22. Unit Capacity Factor (Using DER Net) 1/		
23. Unit Forced Outage Rate		

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
None

25. If Shut Down At End Of Report Period, Estimated Date of Startup: N/A

NO DECISION ON FUTURE OPERATION
SHUTDOWN INDEFINITELY
ALL FUEL UNLOADED.

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
SEE SUMMARY BELOW									

SUMMARY

UNIT SHUTDOWN ON OCTOBER 31, 1974, ALL FUEL HAS BEEN UNLOADED; NO DECISION ON FUTURE OPERATION HAS BEEN MADE.

LEGEND

[1]Type:
F--Forced
S--Scheduled

[2]Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3]Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4]Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5]Exhibit H:
Same source.

INDIAN POINT 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	New York/Westchester 25 miles N of New York City
2.	Electric Reliability Council	NPCC
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2758
5.	Design Electrical Rating (MWE Net)	873
6.	Date of Initial Criticality	May 22, 1973
7.	Date of Commercial Operation	August 1973
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Hudson River

Utility & Contractor Information

10.	Licensee	Consolidated Edison Company of New York, Inc.
11.	Corporate Address	4 Irving Place New York, New York
12.	Corporate Contact	W. J. Cahill, Jr., Vice President
13.	Architect/Engineer	UE&C
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	J. A. Jones/UE&C

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Senior Inspector	R. Keimig
19.	IE Resident Inspector	T. Rebelowski (914-739-9360) FTS 656-9220
20.	Licensing Project Manager	L. Olshan (X27144)
21.	Docket Number	50-247
22.	License Number/Date of Issue	DPR-26, 10/19/71
23.	Public Document Room	White Plains Public Library 100 Martine Avenue White Plains, New York 10601

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

+ The licensee shutdown Unit 2 on 1/11/80 to perform additional inspections on turbine as outlined in IE Information Notice No.79-37.

FACILITY ITEMS (PLANS AND PROCEDURES):

None

MANAGERIAL ITEMS:

+ None

PLANT STATUS:

+ Routine operation until plant shutdown on 1/11/80.

+ LAST IE SITE INSPECTION DATE: 01/05/80

+ INSPECTION REPORT NO: 79-25

SUMMARY

INSPECTION SUMMARY

+ Jun 20-22 & Jul 23-27 (79-11): Routine, unannounced inspection of the licensee's inservice inspection activities and the eddy current examination of the steam generator tubes. This included review of ISI Program and NDE procedures, observation of work in progress and review of ISI data and eddy current examination data. The inspection involved 59 inspector-hours onsite by one regional based NRC inspector. No items of noncompliance were identified.

+ Jul 17-19 (79-13): Routine, unannounced inspection by regional based inspectors of refueling activities, outage maintenance, integrated leak rate test procedure review, and followup on outstanding items. The inspection involved 67.5 inspector-hours onsite by three NRC regional based inspectors. No items of noncompliance or deviations were identified.

+ Jul 24-27 (79-14): Routine, unannounced inspection by a regional based inspector of radiation protection during refueling including licensee action on IE circulars and previous inspection findings, procedures, advance planning and preparation, training, exposure control, respiratory protection, posting and control, radioactive material control and surveys. Upon arrival, a tour of the radiation control area was conducted to observe radiation safety practices at areas where work was in progress. The inspection involved 29 inspector-hours onsite by one NRC regional based inspector. No items of noncompliance were identified.

+ Aug 1-2 (79-15): Routine, unannounced inspection by two regional based inspectors of licensed reactor operator requalification training, general employee training, craft and technician training, and licensee action on previous inspection findings. The inspection involved 29 inspector-hours onsite by two regional based inspectors. No items of noncompliance were identified.

REPORTS RECEIVED FROM LICENSEE

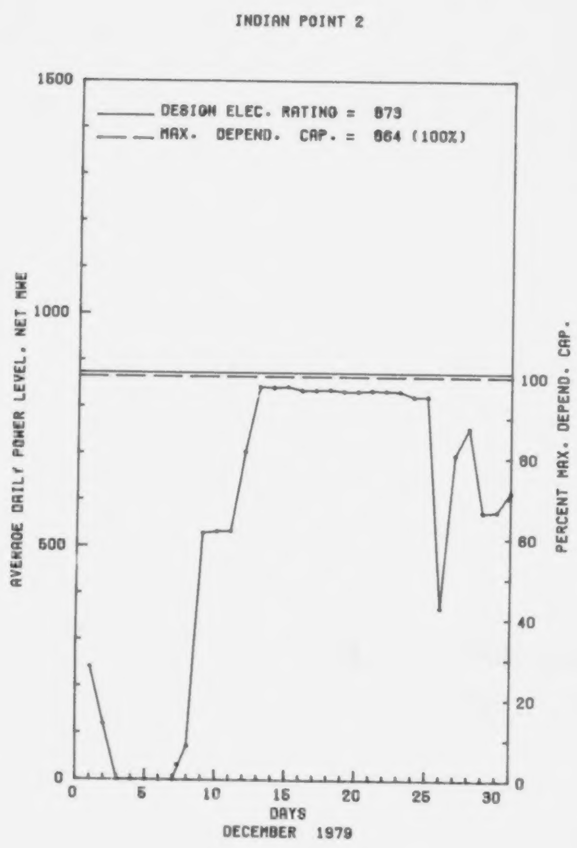
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
NONE				

REVIEWED BY OMPA LOEB: T.C. CINTULA DATE: 2011080

REVIEWED BY NRR DOR: DATE: 2 13 80

INDIAN POINT 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

ITEM 21 CALCULATED BY USING A WEIGHTED AVERAGE

1. Docket Number: 50-247
2. Utility Contact: L. Kawula (914) 694-6000 @ I.P.
3. Licensed Thermal Power (MWt): 2758
4. Nameplate Rating (Gross MWe): 1126 x 0.9 = 1013
5. Design Electrical Rating (Net MWe): 873
6. Maximum Dependable Capacity (Gross MWe): 900
7. Maximum Dependable Capacity (Net MWe): 864
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NO. 6 & 7 Winter Ratings
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	8,760.0	48,241.0
12. Hours Reactor Was Critical	591.2	6,349.2	32,294.5
13. Reactor Reserve Shutdown Hours	.0	119.8	241.4
14. Hours Generator On-Line	575.9	6,160.1	31,389.2
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,368,685	16,029,333	80,832,526
17. Gross Electrical Energy (MWH)	431,520	5,033,650	25,133,086
18. Net Electrical Energy (MWH)	410,795	4,804,928	23,965,100
19 Unit Service Factor	77.4	70.3	65.1
20. Unit Availability Factor	77.4	70.3	65.1
21. Unit Capacity Factor (MDC Net)	63.9	64.0	57.7
22. Unit Capacity Factor (DER Net)	63.2	62.8	56.9
23. Unit Forced Outage Rate	21.6	4.7	8.1

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): TURBINE INSPECTION - 01/11/80.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
28	11/27/79	S	9.8	B	4		HA	PIPEXX	UNIT SHUTDOWN TO REPAIR MISC. STEAM LEAKS AND REPLACE MOTORS ON CRDM COOLING FANS.
29	12/01/79	F	2.8	A	3		CH	VALVEX	UNIT TRIP DUE TO FEEDWATER REGULATORS. - LOW LEVEL IN NO. 23 S/G
30	12/02/79	F	156.3	A	1	79-024	XX	VALVEX	UNIT SHUTDOWN TO INSPECT RELIEF VALVES.
N/A	12/25/79	F	.0	A	5		HF	HTEXCH	LOAD REDUCTION FOR CONDENSER INSPECTION.
N/A	12/28/79	S	.0	D	5		HA	TURBIN	LOAD REDUCTION CAUSED BY L.P. TURBINE RESTRICTION.

SUMMARY

THE UNIT WAS RETURNED TO SERVICE ON 12/01/79 FROM AN OUTAGE TO REPLACE CONTROL ROD DRIVE MECHANISM COOLING FAN MOTORS. AN OUTAGE OCCURRED AGAIN FROM 12/02/79 THROUGH 12/07/79 TO DETERMINE THE SOURCE OF EXCESSIVE LEAKAGE INTO THE PRESSURIZER RELIEF TANK. FROM 12/28/79 THROUGH THE END OF THE PERIOD, POWER WAS REDUCED TO 800 MWE AND THE LOW PRESSURE STEAM DUMP SYSTEM WAS PLACED IN SERVICE BECAUSE OF LOW PRESSURE TURBINE DISC CRACKING AT SIMILAR WESTINGHOUSE UNITS. MAXIMUM POWER WILL BE MAINTAINED AT 800 MWE UNTIL THE SCHEDULED TURBINE INSPECTION ON 01/11/80.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

INDIAN POINT 3

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	New York/Westchester 25 miles N of New York City
2.	Electric Reliability Council	NPCC
3.	Type of Reactor	PWR
4.	Power Level (MWT)	3025
5.	Design Electrical Rating (MWE Net)	965
6.	Date of Initial Criticality	April 6, 1976
7.	Date of Commercial Operation	August 30, 1976
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Hudson River

Utility & Contractor Information

10.	Licensee	Power Authority of the State of New York, Inc.
11.	Corporate Address	10 Columbus Circle New York, New York
12.	Corporate Contact	G.T. Barry General Manager
13.	Architect/Engineer	UE&C
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	WEDCO

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Senior Inspector	R. Keimig
19.	IE Resident Inspector	T. Rebelowski (914-739-9360) FTS 656-9220
20.	Licensing Project Manager	L. Olshan x27144
21.	Docket Number	50-286
22.	License Number/Date of Issue	DPR-64, 12/12/75
23.	Public Document Room	White Plains Public Library 100 Martine Avenue White Plains, New York 10601

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

None

FACILITY ITEMS (PLANS AND PROCEDURES):

+ Plant heatup for Low Power Physics testing is scheduled for January 22, 1980. It is expected that the plant will be returned to service by February 1, 1980.

MANAGERIAL ITEMS:

Plant startup has been delayed to February 1, 1980.

PLANT STATUS:

+ In refueling outage.

+ LAST IE SITE INSPECTION DATE: 12/22/79

+ INSPECTION REPORT NO: 79-29

SUMMARY

INSPECTION SUMMARY

+ Sep 10-14 & 17-20 (79-20): Routine, unannounced inspection of the QA Program; Design Change/Modification Program; Design Changes/Modifications; QA/QC Administration; Records Program; Document Control Program; and, the Offsite Support Staff. The inspection involved 71 inspector-hours onsite by two NRC inspectors and 51.5 inspector-hours at the corporate offices. Three items of noncompliance were identified in three areas (Infraction - implementing a Technical Specification change without prior NRC approval; Infraction - failure to store single copy records per N45.2.9; Infraction - failure to take corrective action with respect to audit report findings).

+ Oct 30-31 (79-24): Special unannounced inspection by two regional based inspectors of the radiation protection program as it relates to steam generator eddy current testing and explosive tube plugging operations including: training; exposure control, respiratory protection, posting and control; surveys; and, plant tour. One item of noncompliance was found in one area (failure to maintain results of surveys).

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
NONE				

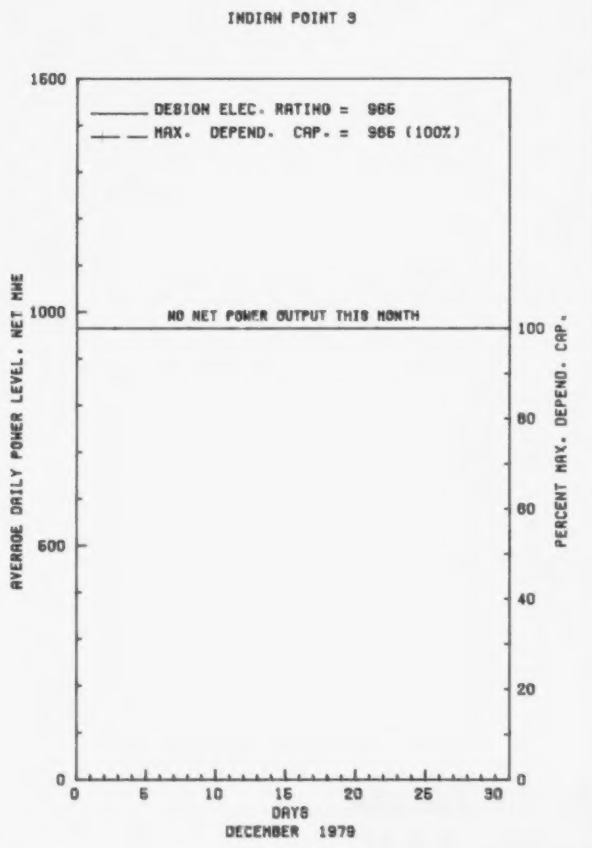
REVIEWED BY OMPA LOEB: T. C. Cintula DATE: 2-11-80

REVIEWED BY NRR DOR: ________ DATE: 2-13-80

GPO 871-887

INDIAN POINT 3

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-286
2. Utility Contact: C. Connell (914) 739-8200 x242
3. Licensed Thermal Power (MWt): 3025
4. Nameplate Rating (Gross MWe): 1126 x 0.9 = 1013
5. Design Electrical Rating (Net MWe): 965
6. Maximum Dependable Capacity (Gross MWe): 1000
7. Maximum Dependable Capacity (Net MWe): 965
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	8,760.0	29,257.0
12. Hours Reactor Was Critical	.0	5,923.9	21,584.9
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	.0	5,825.0	21,032.8
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	15,703,498	56,104,467
17. Gross Electrical Energy (MWH)	0	4,982,330	18,365,101
18. Net Electrical Energy (MWH)	0	4,794,627	17,643,437
19. Unit Service Factor	.0	66.5	71.9
20. Unit Availability Factor	.0	66.5	71.9
21. Unit Capacity Factor (MDC Net)	.0	56.7	62.5
22. Unit Capacity Factor (DER Net)	.0	56.7	62.5
23. Unit Forced Outage Rate	.0	1.6	4.2

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 02/01/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
36	09/14/79	S	744.0	C	4				REFUELING OUTAGE CONTINUES.

SUMMARY

THE UNIT REMAINED SHUTDOWN THROUGHOUT THE REPORT PERIOD FOR REFUELING.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

KEWAUNEE

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Wisconsin/Kewaunee 27 miles E of Green Bay
2.	Electric Reliability Council	N/A
3.	Type of Reactor	PWR
4.	Power Level (MWT)	1650
5.	Design Electrical Rating (MWE Net)	535
6.	Date of Initial Criticality	March 7, 1974
7.	Date of Commercial Operation	June 1974
8.	Condenser Cooling Method	Once through condenser and return to Lake Michigan
9.	Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10.	Licensee	Wisconsin Public Service Corp.
11.	Corporate Address	P. O. Box 1200 Green Bay, Wisconsin 54305
12.	Corporate Contact	E. R. Mathews Power Supply & Eng.
13.	Architect/Engineer	Pioneer Services & Eng. Co.
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Pioneer Services & Eng. Co.

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Senior Inspector	R. F. Warnick
19.	IE Principal Inspector	N. C. Choules
20.	Licensing Project Manager	R. B. Licciardo X28362
21.	Docket Number	50-305
22.	License Number/Date of Issue	DPR-43 issued 12/21/73
23.	Public Document Room	Kewaunee Public Library 822 Juneau Street Kewaunee, Wisconsin 54216

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

The plant operated routinely during the month.

+LAST IE SITE INSPECTION DATE: October 22-26, 1979

+ INSPECTION REPORT NO: 79-19

SUMMARY

INSPECTION SUMMARY

+ Inspection on October 15-19, (79-17): Routine, unannounced inspection of radiation protection program, including: qualifications; audits; training; radiation protection procedures; exposure control; posting, labeling, and control; surveys; notifications and reports; and past inspection items. The inspection involved 32 inspector hours onsite by one NRC inspector. No items of noncompliance or deviations were identified.

+ Inspection on October 22-26, (79-19): Routine, unannounced inspection of plant operations, nonroutine events, noncompliance followup, IE Bulletin and Circular followup and containment purging. The inspection involved 37 inspector hours onsite by one NRC inspector. No items of noncompliance or deviations were identified.

REPORTS RECEIVED FROM LICENSEE

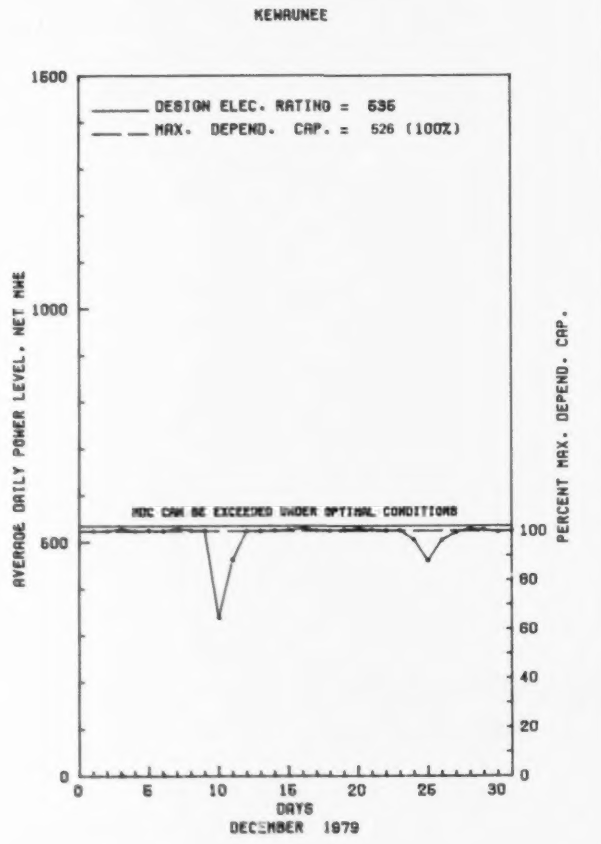
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
NONE				

REVIEWED BY OMPA LOEB: T.C. CINTULA DATE: 2011080

REVIEWED BY NRR DOR: R DATE: 8.14.80.

KEWAUNEE

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

ITEM 21 CALCULATED BY USING A WEIGHTED AVERAGE

1. Docket Number: 50-305
2. Utility Contact: J. Wallace (414) 388-2560 x207
3. Licensed Thermal Power (MWt): 1650
4. Nameplate Rating (Gross MWe): 622 x 0.9 = 560
5. Design Electrical Rating (Net MWe): 535
6. Maximum Dependable Capacity (Gross MWe): 545
7. Maximum Dependable Capacity (Net MWe): 526
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: MDC INCREASED FOR DEC. CAPACITY TESTS
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	8,760.0	48,601.0
12. Hours Reactor Was Critical	742.2	7,038.7	41,002.5
13. Reactor Reserve Shutdown Hours	.0	170.8	2,177.8
14. Hours Generator On-Line	741.1	6,923.7	39,996.0
15. Unit Reserve Shutdown Hours	.0	.0	10.0
16. Gross Thermal Energy (MWH)	1,202,648	10,947,842	61,151,123
17. Gross Electrical Energy (MWH)	400,800	3,613,500	20,183,800
18. Net Electrical Energy (MWH)	382,237	3,439,289	19,199,078
19. Unit Service Factor	99.6	79.0	82.3
20. Unit Availability Factor	99.6	79.0	82.3
21. Unit Capacity Factor (MDC Net)	97.7	75.5	75.3
22. Unit Capacity Factor (DER Net)	96.0	73.4	73.8
23. Unit Forced Outage Rate	.4	11.5	5.3

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NRC REQUIRED DESIGN CHANGES - 01/27/80 - 7 DAYS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
15	12/10/79	F	2.9	G	3		IA	INSTRU	DURING PERFORMANCE OF A MONTHLY SURVEILLANCE TEST AN ERROR WAS MADE WHICH CAUSED A TURBINE RUNBACK. FROM A 100% ARO CONDITION, INSUFFICIENT ROD WORTH WAS AVAILABLE FAST ENOUGH TO PREVENT OVERTEMPERATURE DELTA T TRIP.

SUMMARY

THE UNIT'S MAXIMUM DEPENDABLE CAPACITY (NET MWE) HAS INCREASED FROM 524 TO 526, AS A RESULT OF THE 12/79 CAPACITY TEST. OPERATION WAS AT NEAR FULL POWER THROUGHOUT THE REPORT PERIOD, WITH A REACTOR TRIP ON 12/10/79 DUE TO PERSONNEL ERROR DURING A MONTHLY SURVEILLANCE TEST. BRIEF REDUCTIONS WERE NECESSARY FOR MAINTENANCE.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram.
- 3--Automatic Scram.
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

LACROSSE

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Wisconsin/Vernon 19 miles S of LaCrosse
2.	Electric Reliability Council	MARCA
3.	Type of Reactor	BWR
4.	Power Level (MWT)	165
5.	Design Electrical Rating (MWE Net)	50
6.	Date of Initial Criticality	July 11, 1967
7.	Date of Commercial Operation	November 1, 1969
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Mississippi River

Utility & Contractor Information

10.	Licensee	Dairyland Power Cooperative
11.	Corporate Address	2615 East Avenue South La Crosse, Wisconsin
12.	Corporate Contact	F. Linder
13.	Architect/Engineer	Sargent & Lundy
14.	Nuclear Steam Supply System	Allis-Chalmers
15.	Constructor	Maxson Construction

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	D. C. Boyd
18.	IE Senior Inspector	R. F. Warnick
19.	IE Principal Inspector	K. Ridgway
20.	Licensing Project Manager	J. J. Shea
21.	Docket Number	50-409
22.	License Number/Date of Issue	DPR-45 issued 8/28/73
23.	Public Document Room	LaCrosse Public Library 800 Main Street LaCrosse, Wisconsin 54601

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

MANAGERIAL ITEMS:

+ R. Prince, Radiation Protection Engineer, terminated employment on November 1, 1979. (79-21)

PLANT STATUS

+ Plant is operating routinely.

+LAST IE SITE INSPECTION DATE: October 29 through November 2, 1979

+ INSPECTION REPORT NO: 79-21

INSPECTION SUMMARY

+ Investigation on September 11-12, (79-18): Special, announced investigation of Licensee Event Report 79-16, bypassing of low flow safety circuit during reactor startup on September 7-9, 1979; review of pertinent records, observation of control room conditions, interviews of personnel. The investigation involved 28 inspector hours onsite by two NRC personnel. Of the areas investigated, two items of noncompliance with NRC regulations were observed failure to energize the low flow scram circuit during startup and failure to perform a timely review and corrective action for a previous incident report.

+ Inspection on October 18, (79-20): Annual review meeting for the discussion of the licensee's regulatory performance and areas of NRC concern. The meeting was attended by four NRC personnel and required two hours. No items of noncompliance or deviations were identified.

+ Inspection on October 29 through November 2, (79-21): Routine, unannounced inspection of (1) Environmental Protection Program including: management controls; quality control of analytical measurements; implementation of environmental monitoring program; and (2) Confirmatory Measurements Program including: discussion of results of comparative analyses of previous radiological effluent samples; collection of effluent samples for subsequent comparative analyses. The inspection involved 17 inspector hours onsite by one NRC inspector. No apparent items of noncompliance or deviations were identified.

REPORTS RECEIVED FROM LICENSEE

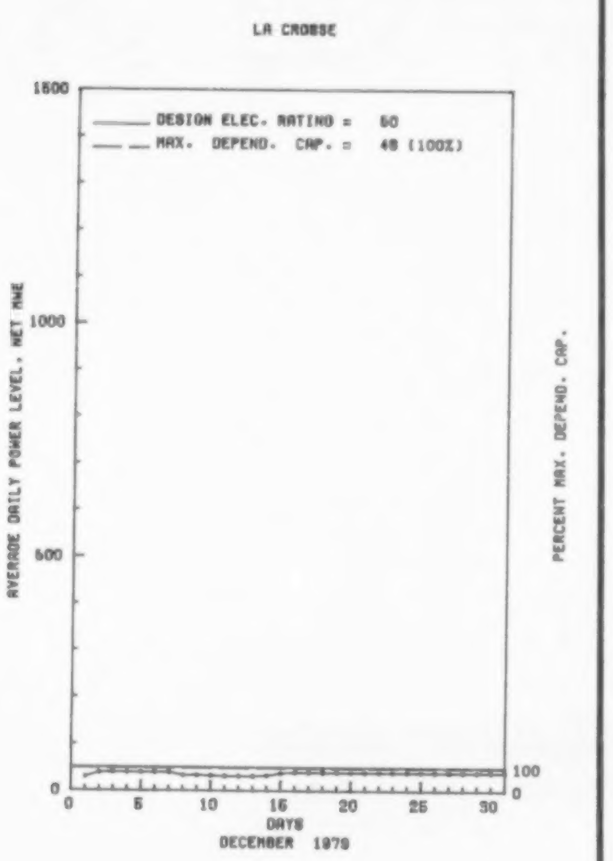
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
NONE				

REVIEWED BY OMPA LOEB: John J. Ryjo DATE: 2-11-80

REVIEWED BY NRR DOR: ______ DATE: ______

LACROSSE

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-409
2. Utility Contact: L. S. Goodman (608) 689-2331
3. Licensed Thermal Power (MWt): 165
4. Nameplate Rating (Gross MWe): 76.8 x 0.85 =65
5. Design Electrical Rating (Net MWe): 50
6. Maximum Dependable Capacity (Gross MWe): 50
7. Maximum Dependable Capacity (Net MWe): 48
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	8,760.0	89,115.0
12. Hours Reactor Was Critical	744.0	6,626.3	57,806.1
13. Reactor Reserve Shutdown Hours	.0	.0	478.0
14. Hours Generator On-Line	744.0	6,286.8	53,005.0
15. Unit Reserve Shutdown Hours	.0	.0	79.0
16. Gross Thermal Energy (MWH)	99,092	747,538	7,284,779
17. Gross Electrical Energy (MWH)	29,325	216,823	2,199,132
18. Net Electrical Energy (MWH)	27,643	200,932	2,032,735
19. Unit Service Factor	100.0	71.8	59.5
20. Unit Availability Factor	100.0	71.8	59.6
21. Unit Capacity Factor (MDC Net)	77.4	47.8	47.5
22. Unit Capacity Factor (DER Net)	74.3	45.9	45.6
23. Unit Forced Outage Rate	.0	12.5	6.4

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUELING - FEBRUARY 5, 1980, - 4 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type^1	Duration (Hours)	Reason^2	Method of Shutting Down Reactor^3	Licensee Event Report No.	System Code^4	Component Code^5	Cause & Corrective Action to Prevent Recurrence
	NONE								

SUMMARY

OPERATION WAS ROUTINE THROUGHOUT THE REPORT PERIOD IN AN END-OF-CYCLE COASTDOWN FOR THE UNITS 02/80 REFUELING OUTAGE.

LEGEND

^{1}Type:
F--Forced
S--Scheduled

^{2}Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

^{3}Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

^{4}Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

^{5}Exhibit H:
Same source.

MAINE YANKEE

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Maine/Lincoln 10 miles N of Bath (near Wiscasset)
2.	Electric Reliability Council	NPCC
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2630
5.	Design Electrical Rating (MWE Net)	825
6.	Date of Initial Criticality	October 23, 1972
7.	Date of Commercial Operation	December 28, 1972
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Back River (Atlantic)

Utility & Contractor Information

10.	Licensee	Maine Yankee Atomic Power Co.
11.	Corporate Address	20 Turnpike Road Westboro, Massachusetts 01581
12.	Corporate Contact	D. W. Edwards, Assistant to Vice President
13.	Architect/Engineer	Stone & Webster
14.	Nuclear Steam Supply System	Combustion Engineering
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	IE Senior Inspector	R. Keimig
19.	IE Principal Inspector	W. J. Lazarus
20.	Licensing Project Manager	M. B. Fairtile
21.	Docket Number	50-309
22.	License Number/Date of Issue	DPR-36, 9/15/72
23.	Public Document Room	Wiscasset Public Library High Street Wiscasset, Maine 04578

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

None

FACILITY ITEMS (PLANS AND PROCEDURES):

- An eight week refueling/maintenance outage is scheduled to commence January 11, 1980.

MANAGERIAL ITEMS:

None

PLANT STATUS:

- Routine operations, EOC coastdown.

- LAST IE SITE INSPECTION DATE: 12/19/79
- INSPECTION REPORT NO: 79-21

SUMMARY

INSPECTION SUMMARY

- Oct 16-19, 22-26 & 29-31 (79-16): Routine, unannounced inspection of QA Program implementation including: program changes; QA/QC organization; design changes/modifications; document control, including drawings; and licensee action on previous inspection findings. The inspection involved 40 hours on site and 40 hours at the corporate offices by two region-based inspectors. No items of noncompliance were identified.

REPORTS RECEIVED FROM LICENSEE

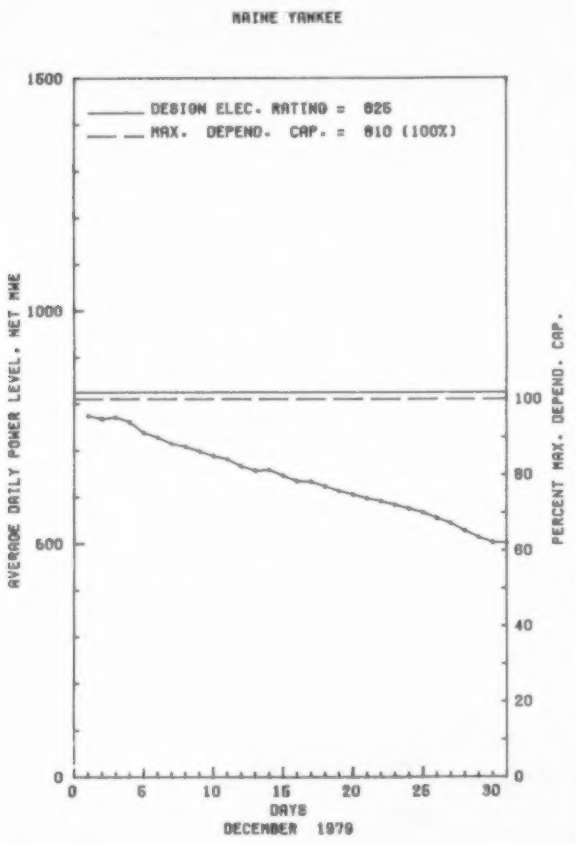
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-027/03L	10/29/79	11/21/79	30 Day LER	Leaking "O" Ring on Safety Injection Tank Check Valve
79-028/03L	10/31/79	11/28/79	30 Day LER	RPS matrix relay failures

REVIEWED BY OMPA LOEB: ______ DATE: 02/11/80

REVIEWED BY NRR DOR: M. B. Fairtile DATE: 2/14/80

MAINE YANKEE

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

CAPACITY FACTORS
CALCULATED WITH A
WEIGHTED AVERAGE

1. Docket Number: 50-309
2. Utility Contact: R. M. Sjogren (617) 366-9011
3. Licensed Thermal Power (MWt): 2630
4. Nameplate Rating (Gross MWe): 900 x 0.9 = 810
5. Design Electrical Rating (Net MWe): 825
6. Maximum Dependable Capacity (Gross MWe): 850
7. Maximum Dependable Capacity (Net MWe): 810
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	8,760.0	62,628 6
12. Hours Reactor Was Critical	744.0	6,166.2	51,20
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	744.0	5,994.5	49,517.7
15. Unit Reserve Shutdown Hours	0	.0	.0
16. Gross Thermal Energy (MWH)	1,501,608	14,117,722	105,232,083
17. Gross Electrical Energy (MWH)	503,100	4,705,370	34,573,180
18. Net Electrical Energy (MWH)	475,684	4,539,015	32,827,186
19. Unit Service Factor	100.0	68.4	79.1
20. Unit Availability Factor	100.0	68.4	79.1
21. Unit Capacity Factor (MDC Net)	78.9	64.0	64.7
22. Unit Capacity Factor (DER Net)	77.5	62.8	63.5
23. Unit Forced Outage Rate	.0	25.5	6.9

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUELING - 01/11/80.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	NONE								

SUMMARY

THE UNIT OPERATED IN AN END-OF-CYCLE COASTDOWN IN PREPARATION FOR THE 01/80 REFUELING OUTAGE.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram.
- 3--Automatic Scram.
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

MILLSTONE 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Connecticut/New London 5 miles SW of New London
2.	Electric Reliability Council	NPCC
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2011
5.	Design Electrical Rating (MWE Net)	690
6.	Date of Initial Criticality	October 26, 1970
7.	Date of Commercial Operation	March 1971
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Long Island Sound

Utility & Contractor Information

10.	Licensee	Northeast Nuclear Energy Company
11.	Corporate Address	P. O. Box 270 Hartford, Connecticut
12.	Corporate Contact	W. G. Counsil, Vice President
13.	Architect/Engineer	Ebasco
14.	Nuclear Steam Supply System	GE
15.	Constructor	Ebasco

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	IE Senior Inspector	E. McCabe (FTS: 244-2000)
19.	IE Resident Inspector	T. Shedlosky (203) 447-3179
20.	Licensing Project Manager	J. J. Shea
21.	Docket Number	50-245
22.	License Number/Date of Issue	DPR-21, 10/7/70
23.	Public Document Room	Waterford Public Library Rope Ferry Road Route 156 Waterford, Connecticut 06385

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

None

FACILITY ITEMS (PLANS AND PROCEDURES):

None

MANAGERIAL ITEMS:

None

PLANT STATUS:

* Routine full power operations. Representative off-gas and stack release rates are 120,000 and 500 microcuries per second, respectively.

* LAST IE SITE INSPECTION DATE: 12/24/79

* INSPECTION REPORT NO: 79-30

SUMMARY

INSPECTION SUMMARY

* Sep 2 - Oct 2 (79-22): Routine, onsite regular and backshift inspection by the resident inspector (40 hours) of the control room, accessible portions of the reactor, turbine, radioactive waste, gas turbine generator and intake buildings; radiation protection; physical security; fire protection; plant operating records; surveillance testing; calibration; maintenance; core power distribution limits; and reports to the NRC. Noncompliances: None.

* Oct 15 - Nov 30 (79-28): Routine, onsite regular, backshift and weekend inspection by the resident inspector (48 hours) of the control room, accessible portions of the reactor, turbine, radioactive waste, gas turbine generator and intake buildings; radiation protection; physical security; fire protection; plant operating records; surveillance testing; calibration; maintenance; core power distribution limits; and reports to the NRC. Noncompliances: None.

REPORTS RECEIVED FROM LICENSEE

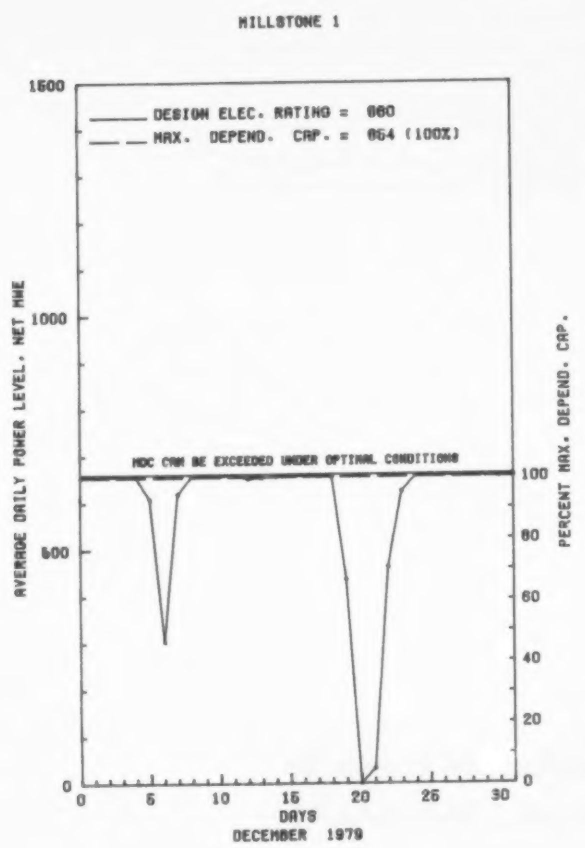
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-031/03L	10/16/79	11/15/79	30 Day LER	FWCI subsystem inoperable to install two seismic restraints
79-032/03L	11/06/79	12/03/79	30 Day LER	Two of four high drywell pressure switch setpoints found slightly above TS limits
79-033/03L	11/13/79	12/04/79	30 Day LER	One of sixteen main steam line high flow switches found set higher than TS limits
79-034/03L	11/15/79	12/14/79	30 Day LER	Failure of LPCI inboard isolation valve which resulted in the loss of ability to inject LPCI to B recirculation loop

REVIEWED BY OMPA LOEB: John J. Riggs DATE: 2-11-80

REVIEWED BY NRR DOR: DATE: [illegible]

MILLSTONE 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-245
2. Utility Contact: R. Young (203) 447-1792 x475
3. Licensed Thermal Power (MWt): 2011
4. Nameplate Rating (Gross MWe): 735 x 0.9 = 662
5. Design Electrical Rating (Net MWe): 660
6. Maximum Dependable Capacity (Gross MWe): 684
7. Maximum Dependable Capacity (Net MWe): 654
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	8,760.0	79,680.0
12. Hours Reactor Was Critical	701.0	6,898.0	60,385.0
13. Reactor Reserve Shutdown Hours	43.0	43.0	935.0
14. Hours Generator On-Line	692.4	6,775.6	58,095.9
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,334,437	12,994,657	104,047,945
17. Gross Electrical Energy (MWH)	456,200	4,426,300	35,231,396
18. Net Electrical Energy (MWH)	435,637	4,221,264	33,639,782
19. Unit Service Factor	93.1	77.3	72.9
20. Unit Availability Factor	93.1	77.3	72.9
21. Unit Capacity Factor (MDC Net)	89.5	73.7	64.6
22. Unit Capacity Factor (DER Net)	88.7	73.0	64.0
23. Unit Forced Outage Rate	6.9	5.8	16.6

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
15	12/05/79	S	.0	B	5		HC	HTEXCH	REDUCED POWER FOR MAIN CONDENSER MAINTENANCE. POWER WAS INCREASED FOLLOWING MAINTENANCE.
16	12/19/79	F	51.6	A	3				UNIT INADVERTENTLY REMOVED FROM SERVICE BY TURBINE TRIP/REACTOR SCRAM DUE TO LOSS OF MAIN GENERATOR LOSS OF EXCITATION. THE UNIT WAS RETURNED TO SERVICE FOLLOWING REPAIRS TO THE AMPLIDYNE GENERATOR.

SUMMARY

OPERATION WAS AT NEAR FULL POWER DURING THE REPORT PERIOD, WITH A POWER REDUCTION ON 12/05/79 FOR MAIN CONDENSER MAINTENANCE. ON 12/19/79, THE UNIT WAS SHUTDOWN BY A TURBINE TRIP/REACTOR SCRAM DUE TO MAIN GENERATOR LOSS OF EXCITATION.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

MILLSTONE 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Connecticut/New London 5 miles SW of New London
2.	Electric Reliability Council	NPCC
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2700
5.	Design Electrical Rating (MWE Net)	870
6.	Date of Initial Criticality	October 17, 1975
7.	Date of Commercial Operation	December 26, 1975
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Long Island Sound

Utility & Contractor Information

10.	Licensee	Northeast Nuclear Energy Company
11.	Corporate Address	P. O. Box 270 Hartford, Connecticut
12.	Corporate Contact	W. G. Counsil, Vice President
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Combustion Engineering
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	IE Senior Inspector	E. McCabe (FTS: 244-2000)
19.	IE Resident Inspector	T. Shedlosky (203) 447-3179
20.	Licensing Project Manager	E. L. Conner, Jr.
21.	Docket Number	50-336
22.	License Number/Date of Issue	DPR-65, 8/1/75
23.	Public Document Room	Waterford Public Library Rope Ferry Road Route 156 Waterford, Connecticut 06385

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO TS 6.8.3, A TEMPORARY PROCEDURE CHANGE WAS MADE TO SP-2001D, REV 1, WITHOUT THE REQUIRED APPROVAL BY 2 MEMBERS OF THE PLANT MANAGEMENT & SUBSEQUENT SORC REVIEW.
(7925 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

None.

FACILITY ITEMS (PLANS AND PROCEDURES):

None

MANAGERIAL ITEMS:

None

PLANT STATUS:

Operating routinely.

LAST IE SITE INSPECTION DATE: 12/24/79

INSPECTION REPORT NO: 79-31

SUMMARY

INSPECTION SUMMARY

Sep 2 - Oct 2 (79-23): Routine, onsite, regular and backshift inspection by the resident inspector (32 hours) of the control room, unit enclosure, auxiliary, turbine and intake buildings, and the condensate polishing facility; radiation protection; physical security; fire protection; plant operating records; surveillance testing; calibration; maintenance; core power distribution limits; and reporting to the NRC. Noncompliances: None.

Oct 15 - Nov 30 (79-29): Routine, onsite, regular backshift and weekend inspection by the resident inspector (112 hours) of: the control room, primary containment enclosure, auxiliary, turbine and intake buildings, and the condensate polishing facility; radiation protection; physical security; fire protection, plant operating records; surveillance testing; calibration maintenance; core power distribution limits; and reporting to the NRC. Noncompliances: None.

REPORTS RECEIVED FROM LICENSEE

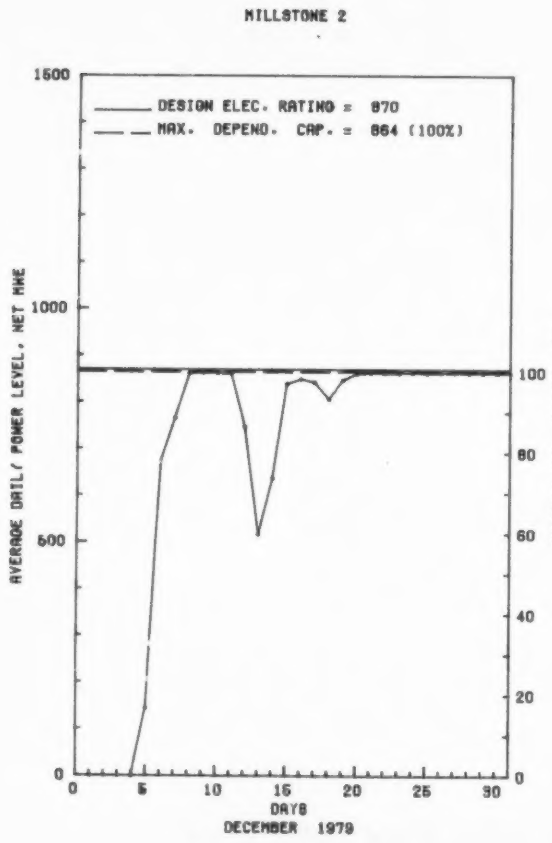
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-034/03L	11/03/79	11/20/79	30 Day LER	Excessive leakage identified outer personnel access hatch. Leak repaired and tested.
79-035/03L	11/02/79	11/28/79	30 Day LER	Channel A high power setpoint found high while reactor was in cold shutdown condition
79-036/03L	11/01/79	11/30/79	30 Day LER	Containment integrity degraded for approximately 30 seconds during plant cooldown
79-037/03L	11/14/79	12/07/79	30 Day LER	Leakage from boric acid evaporator 'B' concentrates pump discharge line due to hairline crack. Leakage contained in the radwaste system
79-038/03L	11/13/79	12/13/79	30 Day LER	Leak on 6" radwaste line due to pitting or cracking. Cause under investigation

REVIEWED BY OMPA LOEB: M R Beebe DATE: 02/11/80

REVIEWED BY NRR DOR: E L Conner DATE: 2/13/80

MILLSTONE 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

ITEMS 21 AND 22 ARE CALCULATED BY USING A WEIGHTED CAPACITY FACTOR

1. Docket Number: 50-336
2. Utility Contact: G. H. Howlett (203) 447-1791 x364
3. Licensed Thermal Power (MWt): 2700
4. Nameplate Rating (Gross MWe): 1011 x 0.9 = 910
5. Design Electrical Rating (Net MWe): 870
6. Maximum Dependable Capacity (Gross MWe): 895
7. Maximum Dependable Capacity (Net MWe): 864
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	8,760.0	35,208.0
12. Hours Reactor Was Critical	651.3	5,551.0	25,078.7
13. Reactor Reserve Shutdown Hours	.0	71.7	2,072.4
14. Hours Generator On-Line	636.3	5,388.5	23,820.2
15. Unit Reserve Shutdown Hours	.0	109.4	335.4
16. Gross Thermal Energy (MWH)	1,613,522	13,816,395	57,698,628
17. Gross Electrical Energy (MWH)	535,134	4,555,844	18,584,645
18. Net Electrical Energy (MWH)	514,155	4,363,567	17,780,258
19. Unit Service Factor	85.5	61.5	67.7
20. Unit Availability Factor	85.5	62.8	68.6
21. Unit Capacity Factor (MDC Net)	80.0	58.5	61.8
22. Unit Capacity Factor (DER Net)	79.4	58.6	60.5
23. Unit Forced Outage Rate	14.5	22.1	23.8

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWN/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
8	10/31/79	F	107.7	H	4		HH	PIPEXX	CONTINUATION OF OUTAGE FROM PREVIOUS MONTH.
9	12/13/79	F	.0	H	5		HH	PUMPXX	REDUCED LOAD TO REPLACE STEAM GENERATOR FEED PUMP 'A' SEAL.

SUMMARY

OPERATION WAS AT NEAR FULL POWER DURING THE REPORT PERIOD, FOLLOWING COMPLETION ON 12/04/79 OF LAST PERIOD'S STEAM GENERATOR SAFE END WELD REPAIR OUTAGE. ON 12/13/79, LOAD WAS REDUCED BRIEFLY TO REPLACE THE STEAM GENERATOR FEED PUMP "A" SEAL.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

MONTICELLO

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Minnesota/Wright 30 miles NW of Minneapolis
2.	Electric Reliability Council	MARCA
3.	Type of Reactor	BWR
4.	Power Level (MWT)	1670
5.	Design Electrical Rating (MWE Net)	545
6.	Date of Initial Criticality	December 10, 1970
7.	Date of Commercial Operation	June 30, 1971
8.	Condenser Cooling Method	Once Through with Cooling Tower
9.	Condenser Cooling Water Source	Mississippi River

Utility & Contractor Information

10.	Licensee	Northern States Power Co.
11.	Corporate Address	414 Nicollet Mall - 8th Floor Minneapolis, Minnesota 55401
12.	Corporate Contact	G. H. Neils (Oper.) L. O. Mayer (Lic.)
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	GE
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heisman
18.	IE Senior Inspector	R. F. Warnick
19.	IE Principal Inspector	N. C. Choules
20.	Licensing Project Manager	T. Kevern
21.	Docket Number	50-263
22.	License Number/Date of Issue	DPR-22 issued 9/8/70
23.	Public Document Room	Environmental Conservation Library Minneapolis Public Library 300 Nicollet Mall Minneapolis, Minnesota 55401

INSPECTION STATUS

ENFORCEMENT STATUS

10 CFR PART 50, APPENDIX B, CRITERION II, STATES THAT "THE (QUALITY ASSURANCE) PROGRAM SHALL PROVIDE FOR INDOCTRINATION AND TRAINING OF PERSONNEL PERFORMING ACTIVITIES AFFECTING QUALITY AS NECESSARY TO ASSURE THAT SUITABLE PROFICIENCY IS ACHIEVED AND MAINTAINED." 1 ACD 5.4 "MATERIAL RECEIPT AND CONTROL" ALSO STATES THAT THOSE PERSONNEL WHO PERFORM INSPECTIONS REQUIRED BY THIS DIRECTIVE SHALL BE QUALIFIED BY APPROPRIATE TRAINING AND EXPERIENCE. SUCH QUALIFICATION SHALL BE DOCUMENTED. CONTRARY TO THE ABOVE IT WAS FOUND THAT NO PROGRAM OR RECORDS OF A PRIOR PROGRAM EXIST EITHER AT THE SITE OR AT THE CORPORATE OFFICE WHICH ADDRESS THE AREA OF QUALITY ASSURANCE TRAINING. (7915 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

FACILITY ITEMS(PLANS AND PROCEDURES):

+ On November 12, 1979, the plant reached full recirculation flow with all control rods fully withdrawn from the core at 100% power. The licensee plans to coastdown in power level until the refueling outage in early February. (79-18)

PLANT STATUS

+ The plant continued in the coastdown mode of operation. Power level at the end of the month was 87%.

+LAST IE SITE INSPECTION DATE: November 13-15, 1979

+ INSPECTION REPORT NO: 79-18

SUMMARY

INSPECTION SUMMARY

+ Inspection on September 10-13, 25-26 and October 17, (79-15): Announced special inspection of the licensee's implementation of the Quality Assurance Program in the following areas: QA/QC administration; qualification of personnel; design changes and modifications; records control; receipt storage and handling; tests and experiments; procurement control; document control; onsite review committees; audits; test and measurement equipment; surveillance and calibration; maintenance and housekeeping and cleanliness. The inspection involved 95 inspector hours onsite by three NRC inspectors. Of the 14 areas inspected, no apparent items of noncompliance or deviations were identified in 13 areas, one apparent item of noncompliance was identified in one area (failure to establish a program for quality assurance training).

+ Inspection or Novmeber 13-15, (79-18): Routine, unannounced inspection of nonroutine event followup, IE Bulletin and Circular followup, onsite review committee activities and containment purging. The inspection involved 21 inspector hours onsite by one NRC inspector. No items of noncompliance or deviations were identified.

REPORTS RECEIVED FROM LICENSEE

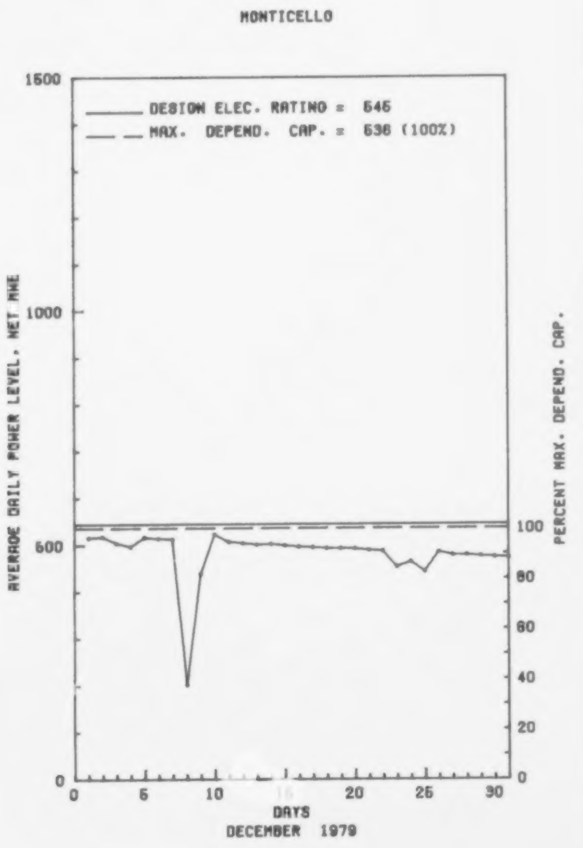
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
	NONE			

REVIEWED BY OMPA LOEB: John J. Rizzo DATE: 2-11-80

REVIEWED BY NRR DOR: T. A. Kevern DATE: 2/13/80

MONTICELLO

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-263
2. Utility Contact: A. L. Myrabo (612) 295-5151
3. Licensed Thermal Power (MWt): 1670
4. Nameplate Rating (Gross MWe): 632 x 0.9 = 569
5. Design Electrical Rating (Net MWe): 545
6. Maximum Dependable Capacity (Gross MWe): 564
7. Maximum Dependable Capacity (Net MWe): 536
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	8,760.0	74,545.0
12. Hours Reactor Was Critical	744.0	8,603.2	61,530.9
13. Reactor Reserve Shutdown Hours	.0	.0	940.7
14. Hours Generator On-Line	744.0	8,551.2	59,970.1
15. Unit Reserve Shutdown Hours	.0	.3	.0
16. Gross Thermal Energy (MWH)	1,097,057	13,463,718	98,942,862
17. Gross Electrical Energy (MWH)	372,620	4,581,450	30,978,060
18. Net Electrical Energy (MWH)	358,158	4,399,560	29,632,204
19. Unit Service Factor	100.0	97.6	80.4
20. Unit Availability Factor	100.0	97.6	80.4
21. Unit Capacity Factor (MDC Net)	89.8	93.7	74.2
22. Unit Capacity Factor (DER Net)	88.3	92.2	72.9
23. Unit Forced Outage Rate	.0	1.8	6.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUELING - FEBRUARY 10, 1980 - 40 DAYS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
17	12/08/79	S	0	B	5		HC	HTEXCH	REDUCED POWER TO 30% TO PLUG CONDENSER TUBE LEAKS AND CLEAN CONDENSER.

SUMMARY

OPERATION WAS ROUTINE DURING THE REPORT PERIOD, IN AN END OF CYCLE COASTDOWN. ON 12/08/79, POWER WAS FURTHER REDUCED TO PLUG CONDENSER TUBE LEAKS AND CLEAN THE CONDENSER.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

NINE MILE POINT 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	New York/Oswego 8 miles NE of Oswego
2.	Electric Reliability Council	NPCC
3.	Type of Reactor	BWR
4.	Power Level (MWT)	1850
5.	Design Electrical Rating (MWE Net)	620
6.	Date of Initial Criticality	September 5, 1969
7.	Date of Commercial Operation	December 1969
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Lake Ontario

Utility & Contractor Information

10.	Licensee	Niagara Mohawk Power Corporation
11.	Corporate Address	300 Erie Boulevard West Syracuse, New York 13202
12.	Corporate Contact	R. Schneider, Vice President Electric Operations
13.	Architect/Engineer	Utility
14.	Nuclear Steam Supply System	GE
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Senior Inspector	E. McCabe
19.	IE Principal Inspector	W. Baunack
20.	Licensing Project Manager	P. Polk
21.	Docket Number	50-220
22.	License Number/Date of Issue	DPR-63, 12/26/74
23.	Public Document Room	State University College of Oswego Penfield Library - Documents Oswego, NY 13126 (315)341-2323

INSPECTION STATUS

ENFORCEMENT STATUS

10 CFR 20.101(B)(1) REQUIRES THAT THE DOSE TO THE WHOLE BODY OF AN INDIVIDUAL IN A RESTRICTED AREA FROM RADIOACTIVE MATERIAL & OTHER SOURCES OF RADIATION IN THE LICENSEE'S POSSESSION SHALL NOT EXCEED 3 REM DURING ANY CALENDAR QUARTER. CONTRARY TO THIS REQUIREMENT, AN INDIVIDUAL RECEIVED A 3.79 REM DOSE TO THE WHOLE BODY IN RESTRICTED AREA DURING THE 2ND CALENDAR QUARTER OF 1979.
(7920 INFRACTION)

TS 6.11, "RADIATION PROTECTION PROGRAM," REQUIRES THAT PROCEDURES FOR PERSONNEL RADIATION PROTECTION BE PREPARED CONSISTENT WITH THE REQUIREMENTS OF 10 CFR PART 20, AND THAT THEY BE APPROVED, MAINTAINED, AND ADHERED TO FOR ALL OPERATIONS INVOLVING PERSONNEL RADIATION EXPOSURE. 1. CONTRARY TO PROCEDURE RP-1, "ACCESS AND RADIOLOGICAL CONTROLS" SECTION 4.3.3 ON 9/27/79, AN INDIVIDUAL NOT ENGAGED IN A SPECIFIC JOB WAS FOUND RESTING IN THE PRONE POSITION ON THE 261' ELEVATION OF THE REACTOR BUILDING. 2. CONTRARY TO PROCEDURE RP-2, "RADIATION WORK PERMIT", SECTION 4.4.1 AN INDIVIDUAL NOT QUALIFIED IN RADIATION PROTECTION WAS PERMITTED TO ACT AS "LEADMAN" ON 4 RWP. CONTRARY TO SECTION 4.7 AN INDIVIDUAL WAS PERMITTED TO EXCEED HIS AUTHORIZED EXPOSURE AND CONTRARY TO SECTION 5.4, INDIVIDUALS FAILED TO ADHERE TO PROTECTIVE MEASURES SPECIFIED IN 14 RWP. 3. CONTRARY TO "RADIATION PROTECTION PROCEDURE" SECTION III.F.3.1.B.1, LAUNDRY WAS FOUND CONTAMINATED IN EXCESS OF LICENSEE LIMITS.
(7920 INFRACTION)

AN INDIVIDUAL WORKED IN THE DRYWELL, A RESTRICTED AREA, DURING THE SECOND CALENDAR QUARTER OF 1979, AND WAS NOT INSTRUCTED IN THE HEALTH PROTECTION PROBLEMS ASSOCIATED WITH EXPOSURE TO RADIOACTIVE MATERIALS OR RADIATION; NOR WAS THE INDIVIDUAL INSTRUCTED IN THE PRECAUTIONS OR PROCEDURES TO MINIMIZE EXPOSURE OR TO OBSERVE WITHIN HIS CONTROL, THE APPLICABLE PROVISIONS OF COMMISSION REGULATIONS AND LICENSES FOR THE PROTECTION OF PERSONNEL FROM EXPOSURE TO RADIATION OR RADIOACTIVE MATERIALS.
(7920 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEM AND COMPONENT PROBLEMS:

None

FACILITY ITEMS (PLANS AND PROCEDURES):

None

MANAGERIAL ITEMS:

+ None

PLANT STATUS:

+ The unit operated continuously at essentially full power with four recirculation pumps. Representative stack and offgas release rates are 10 and 13,000 microcuries per second respectively.

+ LAST IE SITE INSPECTION DATE: 12/10-12/79

+ INSPECTION REPORT NO: 79-26

SUMMARY

INSPECTION SUMMARY

+ No inspections completed during this report period.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

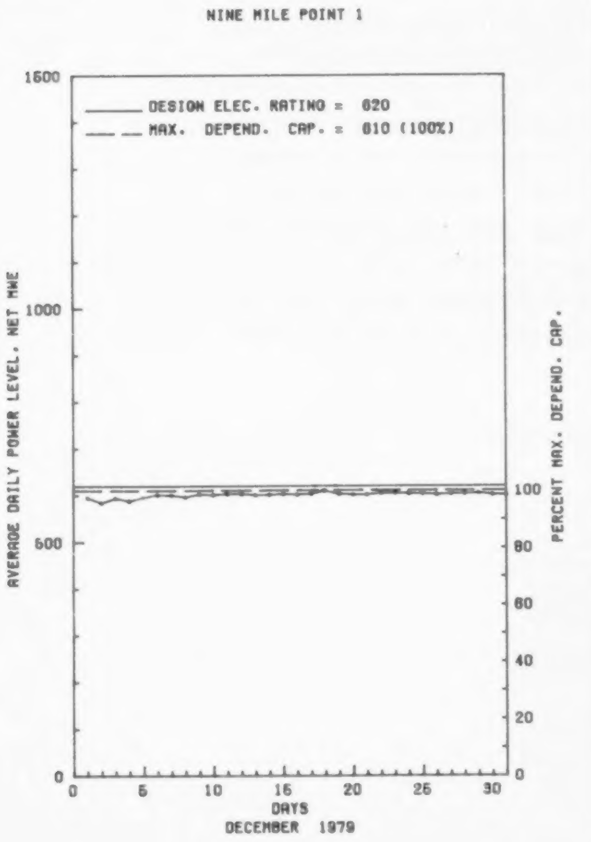
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
NONE				

REVIEWED BY OMPA LOEB: T. C. CINTULA **DATE:** 2011080

REVIEWED BY NRR DOR: Philip J. Polk **DATE:** 2/13/80

NINE MILE POINT 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-220
2. Utility Contact: B. Hawksley (315) 343-2110 x1312
3. Licensed Thermal Power (MWt): 1850
4. Nameplate Rating (Gross MWe): 755 x 0.85 = 642
5. Design Electrical Rating (Net MWe): 620
6. Maximum Dependable Capacity (Gross MWe): 630
7. Maximum Dependable Capacity (Net MWe): 610
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	8,760.0	89,112.0
12. Hours Reactor Was Critical	744.0	5,880.0	65,352.7
13. Reactor Reserve Shutdown Hours	.0	.0	1,204.2
14. Hours Generator On-Line	744.0	5,787.7	62,810.5
15. Unit Reserve Shutdown Hours	.0	.0	20.2
16. Gross Thermal Energy (MWH)	1,352,609	9,669,840	102,024,601
17. Gross Electrical Energy (MWH)	460,871	3,105,324	33,509,510
18. Net Electrical Energy (MWH)	446,453	3,005,389	32,449,765
19. Unit Service Factor	100.0	66.1	70.5
20. Unit Availability Factor	100.0	66.1	70.5
21. Unit Capacity Factor (MDC Net)	98.4	56.2	59.7
22. Unit Capacity Factor (DER Net)	96.8	55.3	58.7
23. Unit Forced Outage Rate	.0	1.0	9.3

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	NONE								

SUMMARY

THE UNIT OPERATED THROUGHOUT THE REPORT PERIOD WITH THE NO. 15 REACTOR RECIRCULATION PUMP OUT OF SERVICE AND ISOLATED DUE TO MECHANICAL PROBLEMS. OPERATION WAS ROUTINE THROUGHOUT THE REPORT PERIOD, WITH BRIEF REDUCTIONS FOR CONTROL ROD ADJUSTMENT.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

NORTH ANNA 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Louisa County, Virginia 40 Miles NW of Richmond, Virginia
2.	Electric Reliability Council	Southeastern
3.	Type of Reactor	PWR
4.	Licensed Power Level (MWT)	2775
5.	Design Electrical Rating (MWE Net)	907
6.	Date of Initial Criticality	April 5, 1978
7.	Date of Commercial Operation	June 6, 1978
8.	Condenser Cooling Method	Once through
9.	Condenser Cooling Water Source	Lake Anna

Utility & Contractor Information

10.	Licensee	Virginia Electric & Power Co.
11.	Corporate Address	P. O. Box 26666 Richmond, Virginia 23261
12.	Corporate Contact	W. L. Proffitt Senior V. P. Power.
13.	Architect/Engineer	Stone & Webster
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. Lewis
18.	IE Senior Inspector	P. J. Kellog
19.	IE Resident Inspector	M. S. Kidd
20.	Licensing Project Manager	L. Engle
21.	Docket Number	50-338
22.	License Number/Date of Issue	NPF-4, 11/26/77
23.	Public Document Room	Alderman Library/Manuscripts Dept. Univ. of Virginia/Charlottesville VA 22901 & Louisa County Courthouse, Louisa, VA 23093

INSPECTION STATUS

ENFORCEMENT STATUS

AS REQUIRED BY TECHNICAL SPECIFICATION 6.8.1.C, WRITTEN PROCEDURES SHALL BE ESTABLISHED IMPLEMENTED AND MAINTAINED COVERING SURVEILLANCE AND TEST ACTIVITIES OF SAFETY RELATED EQUIPMENT. TECHNICAL SPECIFICATION 4.0.5.A.2 REQUIRES INSERVICE TESTING OF ASME CODE CLASS 1, 2 AND 3 VALVES IN ACCORDANCE WITH SECTION XI OF THE ASME BOILER AND PRESSURE VESSEL CODE. SUBSECTION IWV 3400 OF THE ASME BOILER AND PRESSURE VESSEL CODE, SECTION XI REQUIRES CATEGORY A AND B VALVES BE CYCLED EVERY THREE MONTHS TO INSURE OPERABILITY. CONTRARY TO THE ABOVE, CATEGORY A VALVES MOV-RS-100A AND MOV-RS-100B, BOTH SAFETY RELATED COMPONENTS, ARE NOT REQUIRED TO BE CYCLED EVERY THREE MONTHS BY ANY PROCEDURE.
(7941 INFRACTION)

AS REQUIRED BY UNIT 1 TECHNICAL SPECIFICATION TABLE 3.3-3, ITEM 1.E, THREE STEAM PRESSURE CHANNELS PER OPERATING LOOP ARE REQUIRED TO BE OPERABLE IN MODES 1,2 AND 3 EXCEPT AS PERMITTED BY ACTION STATEMENT 14. ACTION STATEMENT 14 STATES THAT WITH THE NUMBER OF OPERABLE CHANNELS ONE LESS THAN THE TOTAL NUMBER OF CHANNELS, OPERATION MAY PROCEED UNTIL PERFORMANCE OF THE NEXT REQUIRED CHANNEL FUNCTIONAL TEST (MONTHLY), PROVIDED THE INOPERABLE CHANNEL IS PLACED IN THE TRIPPED CONDTION WITHIN ONE HOUR. CONTRARY TO THE ABOVE, MAIN STEAM PRESSURE TRANSMITTERS FOR LOOP A, CHANNEL IV (PT-1476) AND LOOP C, CHANNEL IV (PT-1497) WERE INOPERABLE THROUGHOUT THE FIRST CYCLE OF UNIT 1 BECAUSE THEIR SENSING LINES HAD BEEN SWAPPED. THIS WAS APPARENTLY DONE DURING A CUTTING AND REWELDING PROCESS UNDER THE REWORK CONTROL PROGRAM (RCF I-350) IN JULY, 1977.
(7945 INFRACTION)

AS REQUIRED BY UNIT 1 TECHNICAL SPECIFICATION 6.10.2.M, RECORDS OF TRAINING AND QUALIFICATION FOR CURRENT PLANT STAFF MEMBERS ARE TO BE MAINTAINED AS LIFETIME RECORDS. THE NUCLEAR POWER STATION QUALITY ASSURANCE MANUAL SECTION 17 AND SECTION 17.2.14 OF VEPCO'S TOPICAL REPORT - QUALITY ASSURANCE PROGRAM - OPERATIONS PHASE (VEP-1-3A) COMMIT TO RETENTION OF SUCH RECORDS IN ACCORDANCE WITH ANSI N45.2-9, 1974. THIS DOCUMENT REQUIRES A SPECIAL STORAGE REPOSITORY OR DUPLICATE RECORDS SYSTEMS. CONTRARY TO THE ABOVE, RECORDS OF TRAINING RECEIVED BY MECHANICAL AND ELECTRICAL MAINTENANCE EMPLOYEES AS PART OF THE MECHANIC DEVELOPMENT PROGRAMS ARE NOT MAINTAINED AS REQUIRED. THESE RECORDS ARE NOT RETAINED AT THE PLANT SITE AND APPARENTLY ONE COPY IS MAINTAINED IN THE VEPCO CORPORATE OFFICES.
(7945 DEFICIENCY)

AS REQUIRED BY CRITERION V OF APPENDIX B TO 10 CFR 50 IMPLEMENTED BY VEPCO TOPICAL REPORT VEP-1-A SECTION 17, PARAGRAPH 17.2.5 SPECIFIES IN PART, THAT ACTIVITIES AFFECTING QUALITY ARE ACCOMPLISHED IN ACCORDANCE WITH INSTRUCTIONS, PROCEDURES, AND DRAWINGS. GENERAL WELDING PROCEDURE W-1.0 REV. 1 INVOKES ASME SECTION IX (74 W76) WHICH, UNDER PARAGRAPH QW-2013, REQUIRES THAT EACH WELDING PROCESS OR PROCEDURE BE QUALIFIED EITHER SEPARATELY OR IN COMBINATION WITH OTHER PROCESSES OR PROCEDURES...FOR THE DEPOSITED WELD METAL THICKNESS RANGE FOR EACH OF THE PROCESSES OR PROCEDURES...PARAGRAPH QW-351 OF THE SAME CODE REQUIRES THAT THE LIMIT OF THICKNESS FOR WHICH A WELDER IS QUALIFIED IS DEPENDENT UPON THE THICKNESS OF THE WELD HE DEPOSITS WITH EACH WELDING PROCESS. CONTRARY TO THE ABOVE, ON NOVEMBER 16, 1979, PROCEDURE QUALIFICATION RECORDS OF WELD PROCESS SPECIFICATIONS AND WELDER PERFORMANCE QUALIFICATIONS INVOLVING COMBINATION PROCESSES DID NOT INDICATE THE WELD METAL THICKNESS DEPOSITED WITH EACH WELDING PROCESS.
(7947 INFRACTION)

AS REQUIRED BY CRITERION XVII OF APPENDIX B TO 10 CFR 50, AND IMPLEMENTED BY VEPCO TOPICAL REPORT VEP-1-A, SECTION 17 PARAGRAPH 17.2.17 SPECIFIES IN PART THE QUALITY ASSURANCE RECORDS RELATING TO ACTIVITIES AFFECTING QUALITY SHALL BE MAINTAINED AS DOCUMENTARY EVIDENCE OF WORK PERFORMANCE. PARAGRAPH(S) I-2120, I-2130, OF ASME SECTION XI (74, S75) REQUIRES THAT ULTRASONIC EQUIPMENT BE STANDARDIZED FOR SCREEN HEIGHT AND AMPLITUDE CONTROL LINEARITY. CONTRARY TO THE ABOVE, ON NOVEMBER 16, 1979, THERE WAS NO DOCUMENTARY EVIDENCE TO VERIFY THAT CODE REQUIRED STANDARDIZATIONS WERE BEING PERFORMED ON LICENSEE OWNED ULTRASONIC INSTRUMENTS USED FOR VOLUMETRIC EXAMINATIONS.
(7947 DEFICIENCY)

REVIEWED BY OMPA LOEB: T.C. Cintula DATE: 2·11·80

REVIEWED BY NRR DOR: L.B. Engle DATE: 2/13/80

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
10 CFR 21 RPT.	11/14/79	11/28/79		WESTINGHOUSE LOCA-ECCS ANALYSIS NONCONSERVATIVE CONCERNING FUEL ROD BURST CALCULATION PART 21 SERIAL 948.
79-138/ 01T-0	11/08/79	11/21/79	14 DAY LER	WESTINGHOUSE DOES NOT PROVIDE PROCEDURE TO TEST PERMISSIVE CONTACTS INVOLVED IN SAFETY CIRCUIT RESET.
79-141/ 01T-0	11/14/79	11/28/79	14 DAY LER	SAFETY SYSTEM COMPONENTS ARE AFFECTED BY OPERATION CONTAINMENT DEPRESSURIZATION ACTUATION RESET.
79-144/ 03L-0	10/18/79	11/15/79	30 DAY LER	CONTAIMENT INTEGRITY VIOLATED DURING CORE ALTERATIONS.
79-145/ 03L-0	11/06/79	12/04/79	30 DAY LER	RHR SYSTEM REMOVED FROM SERVICE BY INSTALLING A JUMPER IN THE SOLID STATE PROTECTION SYSTEM.
79-146/ 03L-0	11/16/79	12/12/79	30 DAY LER	PRESSURIZER LEVEL PROTECTION CHANNEL III INOPERABLE.
79-147/ 03L-0	10/26/79	11/26/79	30 DAY LER	FIRE HOSE STATIONS INOPERABLE DURING MAINTENANCE.
79-149/ 01T-0	10/31/79	11/15/79	14 DAY LER	SAFEGUARDS AREA EXHAUST SYSTEM FAN DISCHARGE DUCTWORK IS NOT SEISMIC CASE AS REQUIRED BY FSAR.
79-150/ 03L-0	11/01/79	11/29/79	30 DAY LER	SURVEILLANCES NOT RUN ON DC ELECTRIC SYSTEM WEEKLY AS REQUIRED.
79-152/ 01T-0	11/16/79	12/04/79	14 DAY LER	WESTINGHOUSE REPORTS NONCONSERVATISM IN DROPPED ROD ANALYSIS FOR HIGH NEGATIVE POWER RANGE.
79-157/ 01T-0	12/12/79	12/26/79	14 DAY LER	PIPING STRESS ANALYSES - REPORT RESCINDED - ENGINEER FOUND STRESS SATISFACTORY.

PLANT STATUS:

UNIT 1 SHUTDOWN ON SEPTEMBER 25, 1979 FOR THE PRESENT REFUELING MAINTENANCE OUTAGE. OUTAGE IS EXPECTED TO LAST A MINIMUM OF 90 DAYS.

* LAST IE SITE INSPECTION DATE: DECEMBER 21, 1979

* INSPECTION REPORT NO: 50-338/79-49

SUMMARY

INSPECTION SUMMARY

* INSPECTION OCTOBER 22-26 (79-41): THIS ROUTINE UNANNOUNCED INSPECTION INVOLVED 56 INSPECTOR-HOURS ON SITE IN THE AREAS OF REFUELING ACTIVITIES, LICENSEE EVENT REPORTS, AND FOLLOWUP ON PREVIOUSLY IDENTIFIED ITEMS. OF THE THREE AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN TWO AREAS; ONE APPARENT ITEM OF NONCOMPLIANCE WAS FOUND IN ONE AREA (INFRACTION - FAILURE TO IMPLEMENT PROCEDURES TO CONDUCT REQUIRED SURVEILLANCE ON TWO VALVES).

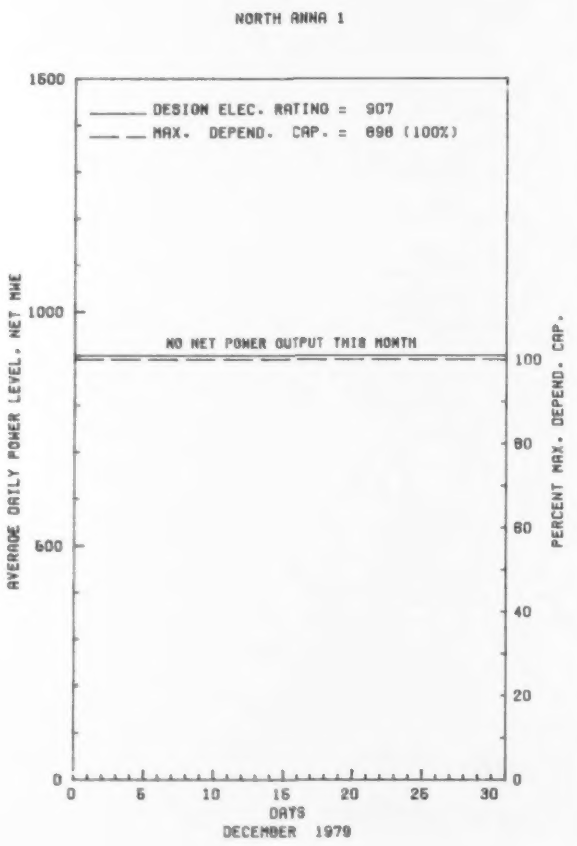
INSPECTION OCTOBER 30 - NOVEMBER 1 (79-43): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 24 INSPECTOR-HOURS ON SITE IN THE AREAS OF INSERVICE INSPECTION ACTIVITY AND IE BULLETIN 79-17. OF THE TWO AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION OCTOBER 9 - NOVEMBER 2 (79-45): THIS ROUTINE INSPECTION BY THE RESIDENT INSPECTOR INVOLVED 37 HOURS ON SITE IN THE AREAS OF LICENSEE EVENT REPORTS, MAINTENANCE PROCEDURES AND ACTIVITIES, PREVIOUSLY IDENTIFIED OPEN AND UNRESOLVED ITEMS, AND PLANT TOURS. WITHIN THE FOUR AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN TWO AREAS. TWO APPARENT ITEMS OF NONCOMPLIANCE WERE IDENTIFIED IN TWO AREAS (INFRACTION - FAILURE TO OPERATE WITHIN THE LIMITING CONDITION FOR OPERATION FOR MAIN STEAM CHANNELS FOR LOOPS A AND C; DEFICIENCY - FAILURE TO MAINTAIN MECHANICAL AND ELECTRICAL MAINTENANCE TRAINING RECORDS IN ACCORDANCE WITH REQUIREMENTS).

INSPECTION NOVEMBER 13-16 (79-46): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED NINE INSPECTOR-HOURS ON SITE IN THE AREAS OF PREVIOUS INSPECTION

NORTH ANNA 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-338
2. Utility Contact: W. R. Madison (703) 894-5151
3. Licensed Thermal Power (MWt): 2775
4. Nameplate Rating (Gross MWe): 1089 x 0.9 = 980
5. Design Electrical Rating (Net MWe): 907
6. Maximum Dependable Capacity (Gross MWe): 928
7. Maximum Dependable Capacity (Net MWe): 898
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	8,760.0	13,777.0
12. Hours Reactor Was Critical	.0	5,478.4	10,165.2
13. Reactor Reserve Shutdown Hours	.0	14.6	14.6
14. Hours Generator On-Line	.0	5,401.4	10,053.1
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	14,083,118	26,279,913
17. Gross Electrical Energy (MWH)	0	4,445,912	8,346,349
18. Net Electrical Energy (MWH)	0	4,188,866	7,853,446
19. Unit Service Factor	.0	61.7	73.0
20. Unit Availability Factor	.0	61.7	73.0
21. Unit Capacity Factor (MDC Net)	.0	53.2	63.5
22. Unit Capacity Factor (DER Net)	.0	52.7	62.8
23. Unit Forced Outage Rate	.0	15.8	10.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 01/10/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
79-16	09/25/79	S	744.0	C	4		RC	FUELXX	REFUELING OUTAGE CONTINUES.

SUMMARY

THE UNIT REMAINED SHUTDOWN THROUGHOUT THE REPORT PERIOD FOR REFUELING.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

OCONEE 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	South Carolina/Oconee 30 miles W of Greenville
2.	Electric Reliability Council	Southeastern
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2568
5.	Design Electrical Rating (MWE Net)	887
6.	Date of Initial Criticality	April 19, 1973
7.	Date of Commercial Operation	July 15, 1973
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Keowee Lake

Utility & Contractor Information

10.	Licensee	Duke Power Company
11.	Corporate Address	422 S. Church Street Charlotte, North Carolina 28242
12.	Corporate Contact	W. O. Parker, Jr.
13.	Architect/Engineer	DPC and Bechtel (Containment Bldg.)
14.	Nuclear Steam Supply System	B&W
15.	Constructor	DPC

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R.C. Lewis
18.	IE Senior Inspector	E.H. Verdery
19.	IE Resident Inspector	F. Jape
20.	Licensing Project Manager	M. Fairtile
21.	Docket Number	50-269
22.	License Number/Date of Issue	DPR-38, 2/6/73
23.	Public Document Room	Oconee County Library 201 S. Spring Street Walhalla, South Carolina 29691

INSPECTION STATUS

ENFORCEMENT STATUS

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

REFUELING OUTAGE.

LAST IE SITE INSPECTION DATE: DECEMBER 3-20, 1979

INSPECTION REPORT NO: 50-269/79-41

SUMMARY

INSPECTION SUMMARY

INSPECTION OCTOBER 29 - NOVEMBER 2 (79-30): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 30 INSPECTOR-HOURS ON SITE IN THE AREAS OF RADIOLOGICAL ENVIRONMENTAL MONITORING PROGRAM INCLUDING MANAGEMENT CONTROL, QUALITY CONTROL OF ANALYTICAL MEASUREMENTS, INSPECTION OF ENVIRONMENTAL MONITORING STATIONS, REVIEW OF ENVIRONMENTAL MONITORING DATA, REVIEW OF RADIOLOGICAL ENVIRONMENTAL MONITORING PROCEDURES AND IMPLEMENTATION OF THE MONITORING PROGRAM. OF THE SIX AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION NOVEMBER 19-21 (79-32): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 18 INSPECTOR-HOURS ON SITE IN THE AREA OF PREVIOUSLY IDENTIFIED QUALITY ASSURANCE ITEMS. OF THE ONE AREA INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION NOVEMBER 30, AND DECEMBER 1 (79-33): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 29 INSPECTOR-HOURS ON SITE IN THE AREA OF EMERGENCY DRILLS. OF THE ONE AREA INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT

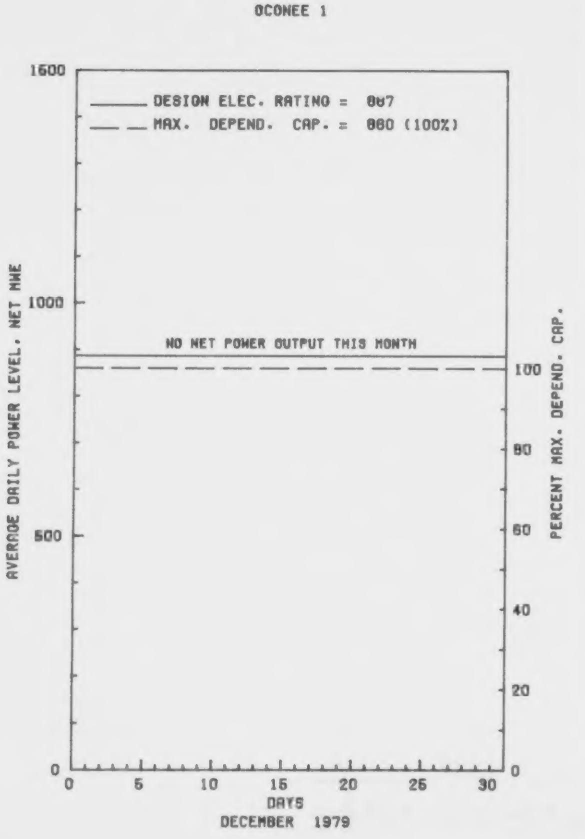
REVIEWED BY OMPA LOEB: PEBobe DATE: 2/11/80

REVIEWED BY NRR DOR: M.B. Fairtile DATE: 2/14/80

best copy available

OCONEE 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

CAPACITY FACTORS
CALCULATED WITH A
WEIGHTED AVERAGE

1. Docket Number: 50-269
2. Utility Contact: J. A. Reavis (704) 373-8552
3. Licensed Thermal Power (MWt): 2568
4. Nameplate Rating (Gross MWe): 1038 x 0.9 = 934
5. Design Electrical Rating (Net MWe): 887
6. Maximum Dependable Capacity (Gross MWe): 899
7. Maximum Dependable Capacity (Net MWe): 860
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	8,760.0	56,641.0
12. Hours Reactor Was Critical	.0	6,286.9	40,514.7
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	.0	6,222.4	37,947.1
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	15,248,956	89,218,119
17. Gross Electrical Energy (MWH)	0	5,273,120	30,914,300
18. Net Electrical Energy (MWH)	-2,853	5,000,177	29,231,499
19. Unit Service Factor	.0	71.0	67.0
20. Unit Availability Factor	.0	71.0	67.0
21. Unit Capacity Factor (MDC Net)	.0	66.4	59.8
22. Unit Capacity Factor (DER Net)	.0	64.4	58.3
23. Unit Forced Outage Rate	.0	18.3	18.0

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 02/02/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
34	11/21/79	S	744.0	C	4		RC	FUELXX	REFUELING CONTINUES. IE BULLETIN 79-02 AND 79-14 CONTINUES.

SUMMARY

THE UNIT REMAINED SHUTDOWN THROUGHOUT THE REPORT PERIOD FOR REFUELING AND PIPE HANGER/SUPPORT INSPECTION AND MODIFICATION PER NRC BULLETIN NOS. 79-02 AND 79-14.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

OCONEE 2

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	South Carolina/Oconee 30 miles W of Greenville
2.	Electric Reliability Council	Southeastern
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2568
5.	Design Electrical Rating (MWE Net)	887
6.	Date of Initial Criticality	November 11, 1973
7.	Date of Commercial Operation	September 9, 1974
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Keowee Lake

Utility & Contractor Information

10.	Licensee	Duke Power Company
11.	Corporate Address	422 S. Church Street Charlotte, North Carolina 28242
12.	Corporate Contact	W. O. Parker, Jr.
13.	Architect/Engineer	DPC/Bechtel
14.	Nuclear Steam Supply System	B&W
15.	Constructor	DPC

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R.C. Lewis
18.	IE Senior Inspector	E.H. Verdery
19.	IE Resident Inspector	F. Jape
20.	Licensing Project Manager	M. Fairtile
21.	Docket Number	50-270
22.	License Number/Date of Issue	DPR-47, 10/6/73
23.	Public Document Room	Oconee County Library 201 S. Spring Street Walhalla, South Carolina 29691

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

POWER OPERATION.

LAST IE SITE INSPECTION DATE: DECEMBER 3-28, 1979

INSPECTION REPORT NO: 50-270/79-39

SUMMARY

INSPECTION SUMMARY

INSPECTION OCTOBER 29 - NOVEMBER 2 (79-28): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 30 INSPECTOR-HOURS ON SITE IN THE AREAS OF RADIOLOGICAL ENVIRONMENTAL MONITORING PROGRAM INCLUDING MANAGEMENT CONTROL, QUALITY CONTROL OF ANALYTICAL MEASUREMENTS, INSPECTION OF ENVIRONMENTAL MONITORING STATIONS, REVIEW OF ENVIRONMENTAL MONITORING DATA, REVIEW OF RADIOLOGICAL ENVIRONMENTAL MONITORING PROCEDURES AND IMPLEMENTATION OF THE MONITORING PROGRAM. OF THE SIX AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION NOVEMBER 19-21 (79-30): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 18 INSPECTOR-HOURS ON SITE IN THE AREA OF PREVIOUSLY IDENTIFIED QUALITY ASSURANCE ITEMS. OF THE ONE AREA INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION NOVEMBER 30, AND DECEMBER 1 (79-31): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 29 INSPECTOR-HOURS ON SITE IN THE AREA OF EMERGENCY DRILLS. OF THE ONE AREA INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

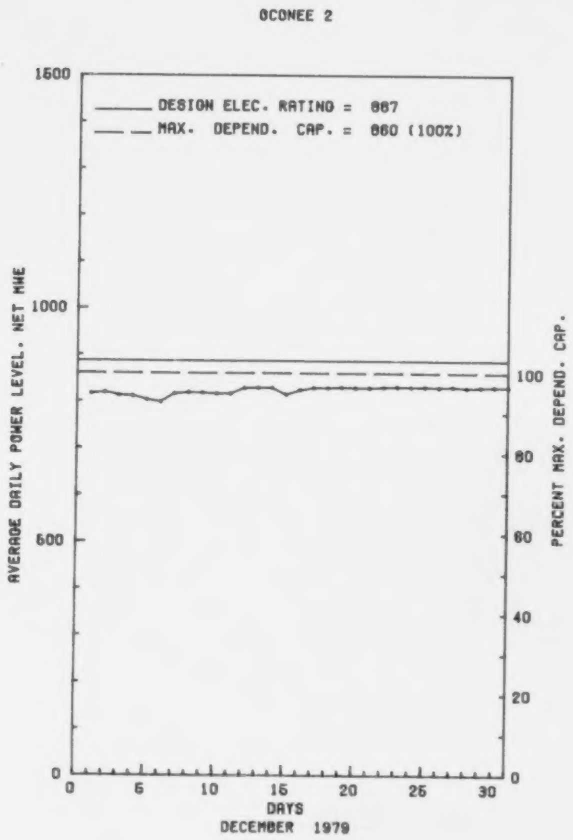
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
NONE				

REVIEWED BY OMPA LOEB: P E Bobe DATE: 2/11/80

REVIEWED BY NRR DOR: M.B Fairtile DATE: 2/14/80

OCONEE 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

CAPACITY FACTORS CALCULATED WITH A WEIGHTED AVERAGE

1. Docket Number: 50-270
2. Utility Contact: J. A. Reavis (704) 373-8552
3. Licensed Thermal Power (MWt): 2568
4. Nameplate Rating (Gross MWe): 1038 x 0.9 = 934
5. Design Electrical Rating (Net MWe): 887
6. Maximum Dependable Capacity (Gross MWe): 899
7. Maximum Dependable Capacity (Net MWe): 860
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	8,760.0	46,561.0
12. Hours Reactor Was Critical	744.0	7,597.4	33,596.0
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	744.0	7,534.9	32,776.
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,887,311	18,555,820	78,047,404
17. Gross Electrical Energy (MWH)	640,150	6,270,810	26,514,356
18. Net Electrical Energy (MWH)	612,390	5,968,288	25,163,758
19. Unit Service Factor	100.0	86.0	70.4
20. Unit Availability Factor	100.0	86.0	70.4
21. Unit Capacity Factor (MDC Net)	95.7	79.2	625
22. Unit Capacity Factor (DER Net)	92.[illegible]	76.8	61.0
23. Unit Forced Outage Rate	.0	10.1	20.0

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUELING - FEBRUARY, 1980 - 12 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
44	12/05/79	F	.0	H	5		HA	FILTER	POWER REDUCTION DUE TO MAXIMUM FLOW THROUGH POLISHING DEMINERALIZER CELLS WITHOUT BYPASSING.

SUMMARY

OPERATION WAS ROUTINE DURING THE REPORT PERIOD, WITH A LOAD REDUCTION ON 12/05/79 DUE TO MAXIMUM FLOW THROUGH POLISHING DEMINERALIZER CELLS WITHOUT BYPASSING.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

OCONEE 3

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	South Carolina/Oconee 30 miles W of Greenville
2.	Electric Reliability Council	Southeastern
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2568
5.	Design Electrical Rating (MWE Net)	887
6.	Date of Initial Criticality	September 5, 1974
7.	Date of Commercial Operation	December 16, 1974
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Keowee Lake

Utility & Contractor Information

10.	Licensee	Duke Power Company
11.	Corporate Address	422 S. Church Street Charlotte, North Carolina 28242
12.	Corporate Contact	W. O. Parker, Jr.
13.	Architect/Engineer	DPC/Bechtel
14.	Nuclear Steam Supply System	B&W
15.	Constructor	DPC

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R.C. Lewis
18.	IE Senior Inspector	E.H. Verdery
19.	IE Resident Inspector	F. Jape
20.	Licensing Project Manager	M. Fairtile
21.	Docket Number	50-287
22.	License Number/Date of Issue	DPR-55, 7/19/74
23.	Public Document Room	Oconee County Library 201 S. Spring Street Walhalla, South Carolina 29691

INSPECTION STATUS

ENFORCEMENT STATUS

AS REQUIRED BY 10 CFR 50, APPENDIX B III, A QUALITY ASSURANCE PROGRAM MUST INCLUDE PROCEDURES TO ASSURE CHANGES IN FACILITY DESIGN ARE REVIEWED BY A DESIGNATED ORGANIZATION. STATION DIRECTIVE 4.4.2. "PROCESSING NUCLEAR STATION MODIFICATIONS" (4/25/79). IMPLEMENTING THIS REQUIREMENT, REQUIRES ALL MODIFICATIONS OF STATION SYSTEMS BE REVIEWED AND IMPLEMENTED IN ACCORDANCE WITH PROCEDURES REFERENCED IN SECTION 6 OF THE STATION DIRECTIVE. THE FINAL SAFETY ANALYSIS REPORT (FSAR), FIGURE 9-3 SHOWS STEAM GENERATOR SAMPLE LINES BEING DRAINED TO THE LIQUID WASTE SYSTEM. FSAR SECTION 11.1.2.2 STATES LOW RADIOACTIVITY LIQUIDS ARE ROUTED TO THE LOC ACTIVITY WASTE TANK. FSAR SECTION 1A.70, DESIGN CRITERIA FOR CONTROL OF RELEASES OF RADIOACTIVITY TO THE ENVIRONMENT, STATES LIQUID WASTES WILL BE MONITORED FOR ACTIVITY LEVEL AT ALL TIMES DURING RELEASE. CONTRARY TO THE ABOVE, ON NOVEMBER 10 AND 11, 1979, RADIOACTIVITY CONTAMINATED LIQUIDS FROM THE STEAM GENERATORS WERE DISCHARGED TO THE LIQUID WASTE SYSTEM. THIS EFFLUENT PATH WAS NOT PROVIDED WITH A RADIATION MONITOR DURING THE RELEASE. THIS CHANGE WAS MADE WITHOUT THE REQUIRED EVALUATION.
(7933 INFRACTION)

AS REQUIRED BY TECHNICAL SPECIFICATION 3.9.3, THE RATE OF RELEASE OF RADIOACTIVE MATERIALS IN LIQUID WASTE FROM THE STATION SHALL BE CONTROLLED SUCH THAT THE INSTANTANEOUS CONCENTRATIONS OF RADIOACTIVITY IN LIQUID WASTE UPON RELEASE FROM THE RESTRICTED AREA, DOES NOT EXCEED THE VALUES LISTED IN 10 CFR 20, APPENDIX B, TABLE II, COLUMN 2. 10 CFR 20.3(14) DEFINES A RESTRICTED AREA AS ANY AREA ACCESS TO WHICH IS CONTROLLED FOR PURPOSES OF PROTECTION OF INDIVIDUALS FROM EXPOSURE TO RADIATION AND RADIOACTIVE MATERIALS. CONTRARY TO THE ABOVE, ON NOVEMBER 10 AND 11, 1979, 176 GALLONS OF WATER WITH A CONCENTRATION OF RADIOACTIVITY TOTALING 3.45 TIMES THE VALUES LISTED IN 10 CFR 20; APPENDIX B, APPENDIX B, TABLE II, COLUMN 2, WERE RELEASED TO THE SITE SEWAGE TREATMENT SUMP. THERE ARE NO CONTROLS OVER THE ACCESS OF INDIVIDUALS IN THIS AREA.
(7933 INFRACTION)

AS REQUIRED BY TECHNICAL SPECIFICATION 6.4.1.M, THE RADIOACTIVE WASTE MANAGEMENT SYSTEM SHALL BE OPERATED IN ACCORDANCE WITH APPROVED PROCEDURES STATION DIRECTIVE 3.8.21, "RADIOACTIVE WASTE DISPOSAL" (6/8/79), SECTION 4.1.1 STATES ALL RADIOACTIVE LIQUIDS MUST BE DISPOSED OF ONLY IN SINKS AND DRAINS THAT GO TO THE LIQUID WASTE DISPOSAL SYSTEM. CONTRARY TO THE ABOVE, ON NOVEMBER 10 AND 11, 1979, 176 GALLONS OF RADIOACTIVITY CONTAMINATED WATER FROM THE A AND B STEAM GENERATORS WAS DRAINED TO THE SEWAGE TREATMENT SYSTEM.
(7933 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

100% POWER OPERATION.

- LAST IE SITE INSPECTION DATE: DECEMBER 3-20, 1979
- INSPECTION REPORT NO: 50-287/79-41

SUMMARY

INSPECTION SUMMARY

- INSPECTION OCTOBER 29 - NOVEMBER 2 (79-30): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 30 INSPECTOR-HOURS ON SITE IN THE AREAS OF RADIOLOGICAL ENVIRONMENTAL MONITORING PROGRAM INCLUDING MANAGEMENT CONTROL, QUALITY CONTROL OF ANALYTICAL MEASUREMENTS, INSPECTION OF ENVIRONMENTAL MONITORING STATIONS, REVIEW OF ENVIRONMENTAL MONITORING DATA, REVIEW OF RADIOLOGICAL ENVIRONMENTAL MONITORING PROCEDURES AND IMPLEMENTATION OF THE MONITORING PROGRAM. OF THE SIX AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION NOVEMBER 19-21 (79-32): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 18 INSPECTOR-HOURS ON SITE IN THE AREA OF PREVIOUSLY IDENTIFIED QUALITY ASSURANCE ITEMS. OF THE ONE AREA INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION OCTOBER 15-16 (79-33): THIS SPECIAL, UNANNOUNCED INSPECTION INVOLVED 17 INSPECTOR-HOURS ON SITE REVIEWING THE DRAINING OF CONTAMINATED LIQUIDS TO THE SEWAGE TREATMENT SYSTEM USING A TEMPORARY DRAIN LINE. THREE APPARENT ITEMS OF NONCOMPLIANCE WERE FOUND IN THIS AREA (INFRACTION - RADIOACTIVE LIQUID RELEASE IN EXCESS OF TECHNICAL SPECIFICATION LIMITS; INFRACTION - FAILURE TO FOLLOW LIQUID WASTE PROCEDURES; INFRACTION - FAILURE TO REVIEW MODIFICATION OF STEAM GENERATOR SAMPLE LINE DRAIN).

INSPECTION NOVEMBER 30, AND DECEMBER 1 (79-34): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 29 INSPECTOR-HOURS ON SITE IN THE AREA OF EMERGENCY DRILLS. OF THE ONE AREA INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

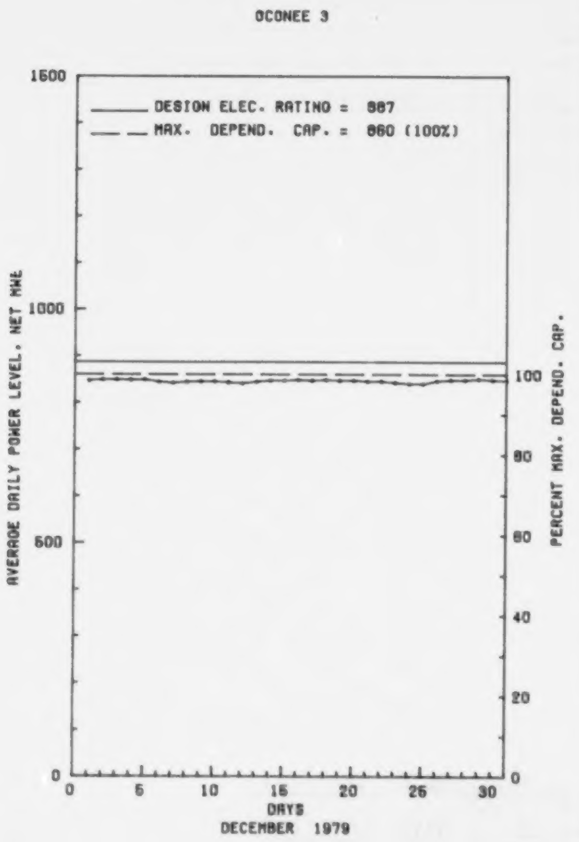
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-012/03L-0	11/07/79	12/07/79	30 DAY LER	LOW PRESSURE SERVICE WATER PUMP INOPERABLE DUE TO SHAFT SLEEVE WORKING LOOSE.

REVIEWED BY OMPA LOEB: PE Bobe **DATE:** 2/11/80

REVIEWED BY NRR DOR: M. B. Fairtile **DATE:** 2/14/80

OCONEE 3

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

CAPACITY FACTORS
CALCULATED WITH A
WEIGHTED AVERAGE

1. Docket Number: 50-287
2. Utility Contact: J. A. Reavis (704) 373-8552
3. Licensed Thermal Power (MWt): 2568
4. Nameplate Rating (Gross MWe): 1038 x 0.9 = 934
5. Design Electrical Rating (Net MWe): 887
6. Maximum Dependable Capacity (Gross MWe): 899
7. Maximum Dependable Capacity (Net MWe): 860
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	8,760.0	44,208.0
12. Hours Reactor Was Critical	744.0	4,127.0	31,895.3
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	744.0	4,040.1	31,061.9
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,898,098	9,858,568	74,381,605
17. Gross Electrical Energy (MWH)	657,200	3,438,770	25,751,264
18. Net Electrical Energy (MWH)	629,295	3,259,529	24,496,556
19. Unit Service Factor	100.0	46.1	70.3
20. Unit Availability Factor	100.0	46.1	70.3
21. Unit Capacity Factor (MDC Net)	98.4	43.3	[illegible].1
22. Unit Capacity Factor (DER Net)	95.4	41.9	62.5
23. Unit Forced Outage Rate	.0	44.4	18.1

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
RELIEF VALVE POSITION INDICATION - 05/80.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	NONE								

SUMMARY

OPERATION WAS ROUTINE THROUGHOUT THE REPORT PERIOD, WITH NO REDUCTIONS OR OUTAGES REPORTED.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

OYSTER CREEK

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	New Jersey/Ocean 9 miles S of Toms River
2.	Electric Reliability Council	MAAC
3.	Type of Reactor	BWR
4.	Power Level (MWT)	1930
5.	Design Electrical Rating (MWE Net)	650
6.	Date of Initial Criticality	May 3, 1969
7.	Date of Commercial Operation	December 1969
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Barnegat Bay

Utility & Contractor Information

10.	Licensee	Jersey Central Power & Light Co.
11.	Corporate Address	Madison Avenue At Punch Bowl Road Morristown, New Jersey 07960
12.	Corporate Contact	I. Finfrock, Vice President Power Generation
13.	Architect/Engineer	Burns & Roe
14.	Nuclear Steam Supply System	GE
15.	Constructor	J. A. Jones

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	IE Senior Inspector	E. McCabe
19.	IE Principal Inspector	L. Briggs
20.	Licensing Project Manager	S. J. Nowicki
21.	Docket Number	50-219
22.	License Number/Date of Issue	DPR-16, 4/9/69
23.	Public Document Room	Ocean County Library 15 Hooper Avenue Toms River, New Jersey 08753

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

None

FACILITY ITEMS (PLANS AND PROCEDURES):

+ The plant was shutdown for a scheduled 10 week refueling outage on January 5.

MANAGERIAL ITEMS:

None.

PLANT STATUS:

+ The plant operated during December with an offgas rate of 160,000 microcuries per second and a stack release rate of 26,700 microcuries per second.

+ LAST IE SITE INSPECTION DATE: 01/14/80

+ INSPECTION REPORT NO: 80-01

SUMMARY

INSPECTION SUMMARY

+ Nov 30 (79-22): Routine, announced inspection by four region-based inspectors (14 hours) of a licensee conducted emergency drill. Noncompliances: None.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

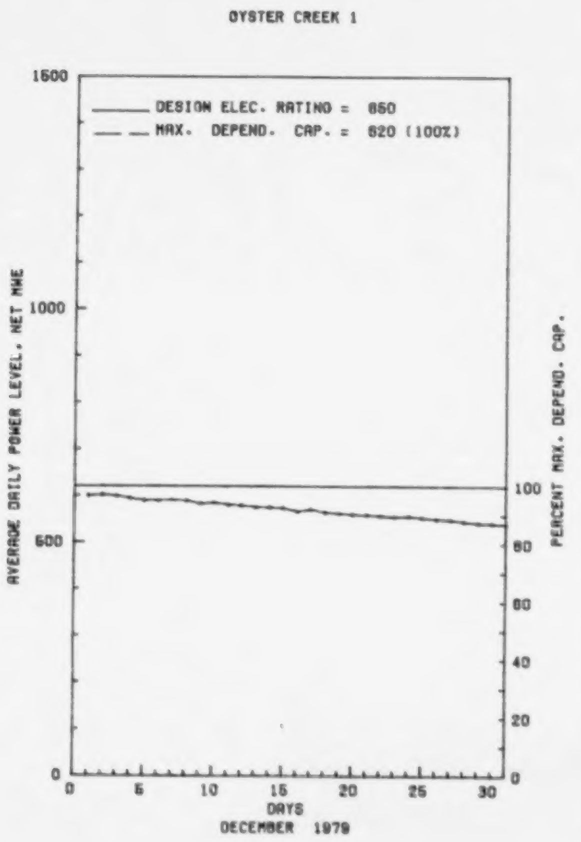
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-037/ 03L	11/03/79	11/30/79	30 Day LER	Failure of core spray booster pump to start during routine surveillance due to control power fuse
79-038/ 03L	11/05/79	11/30/79	30 Day LER	Failure of D.G. No. 1 to start due to position switch adjustment
79-039/ 03L	11/05/79	12/03/79	30 Day LER	APRM Channel 1 (1 of 4) rod block setpoint found 1 percent above T.S. limit
79-040/ 03L	11/02/79	11/30/79	30 Day LER	Failure to perform methyl iodine removal efficiency of SBGTS charcoal filters. Tests completed satisfactorily
79-041/ 03L	11/02/79	12/03/79	30 Day LER	Radioactive releases (low level) from new radwaste building not accurately monitored
79-042/ 03L	11/06/79	12/06/79	30 Day LER	Inadvertent lifting of one electromatic relief valve due to setpoint drift of new pressure switch

REVIEWED BY OMPA LOEB: Jack J. Rizzo DATE: 2-11-80

REVIEWED BY NRR DOR: S. J. Nowicki DATE: 2/12/80

OYSTER CREEK

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

ITEM 21 CALCULATED
BY USING A WEIGHTED
AVERAGE

1. Docket Number: 50-219
2. Utility Contact: C. M. McClain (201) 455-8748
3. Licensed Thermal Power (MWt): 1930
4. Nameplate Rating (Gross MWe): 687.5 x 0.8 = 550
5. Design Electrical Rating (Net MWe): 650
6. Maximum Dependable Capacity (Gross MWe): 650
7. Maximum Dependable Capacity (Net MWe): 620
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	8,760.0	87,854.0
12. Hours Reactor Was Critical	744.0	7,618.2	68,639.3
13. Reactor Reserve Shutdown Hours	.0	.0	468.2
14. Hours Generator On-Line	744.0	7,524.2	67,705.5
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,279,553	13,843,163	113,879,158
17. Gross Electrical Energy (MWH)	440,880	4,742,310	38,882,315
18. Net Electrical Energy (MWH)	423,864	4,563,223	37,481,013
19. Unit Service Factor	100.0	85.9	76.6
20. Unit Availability Factor	100.0	85.9	76.6
21. Unit Capacity Factor (MDC Net)	91.9	84.0	68.8
22. Unit Capacity Factor (DER Net)	87.6	80.1	65.6
23. Unit Forced Outage Rate	.0	14.1	6.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUELING - 01-80
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	NONE								

SUMMARY

OPERATION WAS ROUTINE IN AN END OF CYCLE COASTDOWN THROUGHOUT THE REPORT PERIOD, WITH NO FURTHER REDUCTIONS OR OUTAGES REPORTED.

LEGEND

[1] Type:
F-Forced
S-Scheduled

[2] Reason:
A-Equipment Failure
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error
H-Other

[3] Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Continuations
5-Load Reduction
9-Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

PALISADES

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Michigan/Van Buren 5 miles S of South Haven
2.	Electric Reliability Council	ECAR
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2530
5.	Design Electrical Rating (MWE Net)	805
6.	Date of Initial Criticality	May 24, 1971
7.	Date of Commercial Operation	December 31, 1971
8.	Condenser Cooling Method	Forced Draft Cooling Towers
9.	Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10.	Licensee	Consumers Power Company
11.	Corporate Address	212 W. Michigan Avenue Jackson, Michigan 49201
12.	Corporate Contact	R. B. DeWitt
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Combustion Engineering
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Senior Inspector	D.C. Boyd
19.	IE Resident Inspector	B. Jorgensen
20.	Licensing Project Manager	R. Silver
21.	Docket Number	50-255
22.	License Number/Date of Issue	DPR-20, 3/24/71
23.	Public Document Room	Kalamazoo Public Library 315 South Rose Street Reference Department Kalamazoo, Michigan 49006

INSPECTION STATUS

ENFORCEMENT STATUS

TECHNICAL SPECIFICATION 3.8.1.G REQUIRES A CHECK OF PRIMARY COOLANT SYSTEM BORON CONCENTRATION EACH SHIFT WHEN REFUELING OPERATIONS ARE BEING PERFORMED IN THE REACTOR. CONTRARY TO THE ABOVE, THE LICENSEE FAILED TO CHECK PRIMARY COOLANT SYSTEM BORON CONCENTRATION DURING THE "A" SHIFT ON OCTOBER 2, 1979, A SHIFT DURING WHICH REFUELING OPERATIONS WERE BEING PERFORMED IN THE REACTOR.
(7917 INFRACTION)

TECHNICAL SPECIFICATION 6.8.1.B REQUIRES IMPLEMENTATION OF PROCEDURES COVERING REFUELING OPERATIONS. PROCEDURE FHSO-2 "REFUELING PROCEDURE" INCLUDES CHECKLIST CL A5.14-2 "REFUELING OPERATIONS SHIFT CHECK SHEET," WHICH STATES, IN PART: "THE MINIMUM SPENT FUEL POOL/REACTOR CAVITY LEVEL IS ONE FOOT BELOW THE SURFACE SKIMMERS DURING FUEL HANDLING". CONTRARY TO THE ABOVE, THE LICENSEE HANDLED FUEL ON THE "A" SHIFTS OF OCTOBER 11 AND 12, 1979, WHEN THE WATER LEVEL WAS 1.5 AND 2 FEET, RESPECTIVELY, BELOW THE SKIMMERS.
(7917 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

+ Reactor shutdown on September 18, 1979, to begin a 10-week scheduled refueling and maintenance outage. Start-up tentatively scheduled for late January 1980.

+LAST IE SITE INSPECTION DATE: October 1979

+ INSPECTION REPORT NO: 79-17

SUMMARY

INSPECTION SUMMARY

+ Inspection during October, (79-17): Routine resident inspection program for October 1979, including records; review and audit; maintenance; and refueling preparations and activities. The inspection involved 90 inspector hours onsite. Of the four areas examined, no items of noncompliance or deviations were identified in three areas. Two items of noncompliance (missing required chemistry sampling; and violated procedural limit on refueling water level) were identified in the remaining area.

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT

NONE

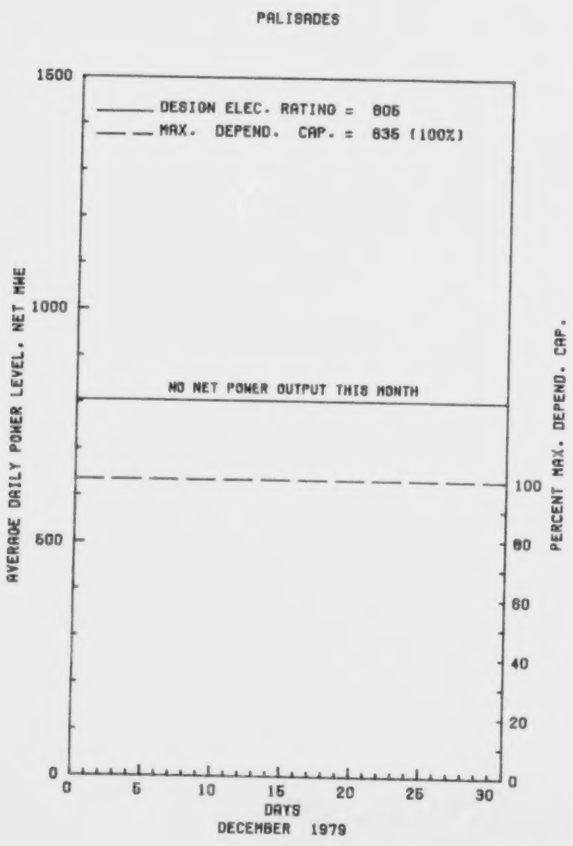
REVIEWED BY UMPA LOEB: Beebe DATE: 02/11/80

REVIEWED BY NRR DOR: DATE: 2/13/80

P-2

PALISADES

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-255
2. Utility Contact: P. J. Bodtke (616) 764-8913
3. Licensed Thermal Power (MWt): 2530
4. Nameplate Rating (Gross MWe): 955 x 0.85 = 812
5. Design Electrical Rating (Net MWe): 805
6. Maximum Dependable Capacity (Gross MWe): 675
7. Maximum Dependable Capacity (Net MWe): 635
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	8,760.0	70,431.0
12. Hours Reactor Was Critical	.0	5,368.9	39,564.8
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	.0	5,244.5	37,426.5
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	12,046,848	71,511,480
17. Gross Electrical Energy (MWH)	0	3,640,590	22,032,230
18. Net Electrical Energy (MWH)	0	3,433,264	20,670,775
19. Unit Service Factor	.0	59.9	53.1
20. Unit Availability Factor	.0	59.9	53.1
21. Unit Capacity Factor (MDC Net)	.0	61.7	46.2
22. Unit Capacity Factor (DER Net)	.0	48.7	36.5
23. Unit Forced Outage Rate	.0	12.7	36.2

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 02/15/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
13	09/08/79	S	744.0	C	4				REFUELING OUTAGE CONTINUES.

SUMMARY

THE UNIT REMAINED SHUTDOWN THROUGHOUT THE REPORT PERIOD FOR REFUELING.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

PEACH BOTTOM 2

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Pennsylvania/York 19 miles S of Lancaster
2.	Electric Reliability Council	MAAC
3.	Type of Reactor	BWR
4.	Power Level (MWT)	3293
5.	Design Electrical Rating (MWE Net)	1065
6.	Date of Initial Criticality	September 16, 1973
7.	Date of Commercial Operation	July 5, 1974
8.	Condenser Cooling Method	Once Through with Cooling Towers
9.	Condenser Cooling Water Source	Conowingo Pond/Susquehanna River

Utility & Contractor Information

10.	Licensee	Philadelphia Electric Company
11.	Corporate Address	2301 Market Street Philadelphia, Pennsylvania 19101
12.	Corporate Contact	S. L. Daltroff, Vice President Electric Production
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	GE
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	IE Senior Inspector	E. McCabe
19.	IE Resident Inspector	E. Greenman 717-456-7614
20.	Licensing Project Manager	Dick Clark
21.	Docket Number	50-277
22.	License Number/Date of Issue	DPR-44, 8/8/73
23.	Public Document Room	Government Publications Section State Library of Pennsylvania Education Building Commonwealth and Walnut Street Harrisburg, Pennsylvania 17126

INSPECTION STATUS

ENFORCEMENT STATUS

10 CFR 50.55A(G)(4), AS OF 8/31/79, SEVERAL SURVEILLANCE TEST PROCEDURES DID NOT CONFORM TO SECTION XI OF THE ASME B&PV CODE. DETAILS ARE AS FOLLOWS: THE HIGH PRESSURE COOLANT INJECTION & REACTOR CORE ISOLATION COOLING PUMPS HAVE VARIABLE SPEED DRIVES & SURVEILLANCE TEST PROCEDURES ST-6.5 & 6.11 DO NOT ESTABLISH A REFERENCE SPEED AS REQUIRED BY IWP-3100.
(7919 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

- Seismic supports for 1 inch CRD insert and withdrawal piping were found to have safety factors for seismic qualification less than required.

FACILITY ITEMS (PLANS AND PROCEDURES):

- An emergency drill involving site evacuation and notification of state, county and local government agencies was conducted on December 16.

MANAGERIAL ITEMS:

PLANT STATUS:

- The reactor operated at a nominal 100 percent power during the month. Representative offgas and combined Unit 2 and Unit 3 stack gas release rates were 8,190 microcuries per second and 959 microcuries per second, respectively. The unit was shutdown on December 31 for repair of a core spray full flow test valve and to implement short term "lessons learned" commitments.

- LAST IE SITE INSPECTION DATE: 12/31/79

- INSPECTION REPORT NO: 79-30

SUMMARY

INSPECTION SUMMARY

- Sep 24-26, & 28 (79-24): Routine, unannounced inspection by one region-based inspector (10 hours onsite) of work activities related to concrete expansion bolt anchors in pipe support base plates under Bulletin 79-02. Noncompliances: None.

- Oct 30 - Nov 1 (79-27): Routine, unannounced inspection by two NRC region-based inspectors (18 hours onsite) of the licensee's response to IE Bulletin 79-19 including maintenance of regulatory requirements, training, audits, procedures and package compliance. Noncompliances: two (failure to adhere to procedures and failure to maintain current procedures at the work location).

- Nov 13-21 (79-28): Routine, unannounced inspection by a region-based inspector (11 hours onsite, 4 hours at corporate offices, 3 hours at maintenance shops) of general employee training, craft personnel training and licensed operator requalification training. Noncompliances: One (failure of operators to review, and/or document review, contents of all abnormal and emergency procedures).

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

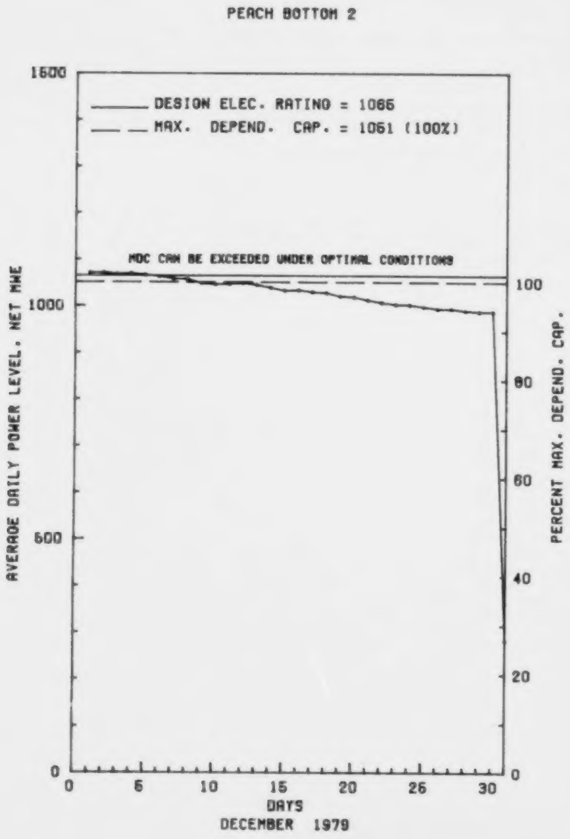
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-046/03L	10/19/79	11/19/79	30 Day LER	D Cad 02 analyzer fuse blew which A analyzer 005
79-047/03L	10/29/79	11/28/79	30 Day LER	Oxygen analyzer primary containment atkomatic solenoid isolation valves fail
79-048/03L	10/30/79	11/29/79	30 Day LER	Main offgas stack isokinetic sample line was broken
79-049/01T	10/05/79	11/16/79	24 Hours	4 seismic supports for 1" CRD pipe inadequate but would not prevent scram. Repair SKED within 7 days
79-050/01T	11/13/79	11/29/79	24 Hours	Trapeze-type hangers for 1" CRD insert/withdraw piping not seismically qualified, but pipe failure would not prevent scram
79-051/01T	11/21/79	12/05/79	24 Hours	Both fire pumps were out of service to install a relief valve modification on the fire header
79-052/01P	11/27/79	12/11/79	24 Hours	Safety factors associated with north and south isolation valve room concrete block wall seismic qualifications

REVIEWED BY OMPA LOEB: T.C. Cintula DATE: 2o11o80

REVIEWED BY NRR DOR: Dick Clark DATE: 02-13-80

PEACH BOTTOM 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-277
2. Utility Contact: W. M. Alden (215) 841-5022
3. Licensed Thermal Power (MWt): 3293
4. Nameplate Rating (Gross MWe): 1280 x 0.9 = 1152
5. Design Electrical Rating (Net MWe): 1065
6. Maximum Dependable Capacity (Gross MWe): 1098
7. Maximum Dependable Capacity (Net MWe): 1051
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	8,760.0	48,144.0
12. Hours Reactor Was Critical	728.0	8,336.0	37,755.0
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	728.0	8,296.5	36,382.8
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	2,307,466	26,544,687	106,765,644
17. Gross Electrical Energy (MWH)	776,680	8,892,300	35,058,230
18. Net Electrical Energy (MWH)	749,337	8,574,430	33,615,785
19. Unit Service Factor	97.8	94.7	76.8
20. Unit Availability Factor	97.8	94.7	76.8
21. Unit Capacity Factor (MDC Net)	95.8	93.1	66.4
22. Unit Capacity Factor (DER Net)	94.6	91.9	65.6
23. Unit Forced Outage Rate	.0	.6	6.3

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
REFUELING, 3/21/80, ELEVEN WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: 1/10/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
13	12/31/79	5	15.0	B	1		CJ	VALVEX	SHUTDOWN FOR REPAIR OF CORE SPRAY FULL-FLOW TEST VALVE.

SUMMARY

OPERATION WAS AT NEAR FULL POWER DURING THE REPORT PERIOD, EXCEPT FOR AN OUTAGE ON 12/31/79 TO REPAIR THE CORE SPRAY FULL-FLOW TEST VALVE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[E] Exhibit H:
Same source.

PEACH BOTTOM 3

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Pennsylvania/York 19 miles S of Lancaster
2.	Electric Reliability Council	MAAC
3.	Type of Reactor	BWR
4.	Power Level (MWT)	3293
5.	Design Electrical Rating (MWE Net)	1065
6.	Date of Initial Criticality	August 7, 1974
7.	Date of Commercial Operation	December 23, 1974
8.	Condenser Cooling Method	Once Through with Cooling Towers
9.	Condenser Cooling Water Source	Conowingo Pond/Susquehanna River

Utility & Contractor Information

10.	Licensee	Philadelphia Electric Company
11.	Corporate Address	2301 Market Street Philadelphia, Pennsylvania 19101
12.	Corporate Contact	S. L. Daltroff, Vice President Electric Production
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	GE
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	IE Senior Inspector	E. McCabe
19.	IE Resident Inspector	E. Greenman (717-456-7614)
20.	Licensing Project Manager	Dick Clark
21.	Docket Number	50-278
22.	License Number/Date of Issue	DPR-56, 7/2/74
23.	Public Document Room	Government Publications Section State Library of Pennsylvania Education Building Commonwealth and Walnut Street Harrisburg, Pennsylvania 17126

INSPECTION STATUS

ENFORCEMENT STATUS

10 CFR 50.55A(G)(4) AS OF 8/31/79 SEVERAL SURVEILLANCE TEST PROCEDURES DID NOT CONFORM TO SECTION XI OF THE ASME B&PV CODE. (7921 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

+ Seismic supports for 1 inch CRD insert and withdrawal piping were found with less than required safety factors for seismic qualification.

+ Unit 3 Tip System shear valves in the Tip Guide Tube Assemblies were installed during initial construction which reversed the positions of the shear and ball valves. Ball valves were qualified in both directions and have been leak checked as a result of this event.

FACILITY ITEMS (PLANS AND PROCEDURES):

+ An emergency drill involving site evacuation including notification of state, county, and local government agencies was conducted on December 16, 1979.

MANAGERIAL ITEMS:

PLANT STATUS:

+ The reactor was operated at a nominal 100 percent power during the month. Routine maintenance and repair on an "A" steam line isolation valve was performed in a scheduled outage December 7-12. One unplanned shutdown occurred on December 14 when the reactor was manually shutdown following a temporary loss of power to certain instrumentation. Representative offgas and combined Unit 2 and Unit 3 stack release rates were 3540 microcuries per second and 959 microcuries per second, respectively.

+ LAST IE SITE INSPECTION DATE: 12/31/79

+ INSPECTION REPORT NO: 79-33

SUMMARY

INSPECTION SUMMARY

+ Sep 24-25, & 28 (79-26): Routine, unannounced inspection by one region-based inspector (10 hours onsite) of work on concrete expansion bolt anchors in pipe support base plates under Bulletin 79-02. Noncompliances: One (failure to initiate corrective action on an identified nonconformance).

+ Oct 16-19 (79-29): Routine, unannounced inspection by a region-based inspector (27 hours onsite) of the inservice inspection program and related nondestructive examination activities, and the documentation associated with capping of the control rod drive hydraulic return line nozzle. Noncompliances: None.

+ Oct 30 - Nov 1 (79-30): Routine, unannounced inspection by two NRC region-based inspectors (18 hours onsite) of the licensee's response to IE Bulletin 79-19 including maintenance of regulatory requirements, training, audits, procedures and package compliance. Noncompliances: Two (failure to adhere to procedures, and failure to maintain current procedures at the work location).

+ Nov 13-21 (79-31): Routine, unannounced inspection by a region-based inspector (11 hours onsite, 3 hours at maintenance ships, 4 hours at corporate offices) of general employee training, craft personnel training and licensed operator requalification training. Noncompliances: One (failure of operators to review, and/or document review, contents of all abnormal and emergency procedures).

REPORTS RECEIVED FROM LICENSEE

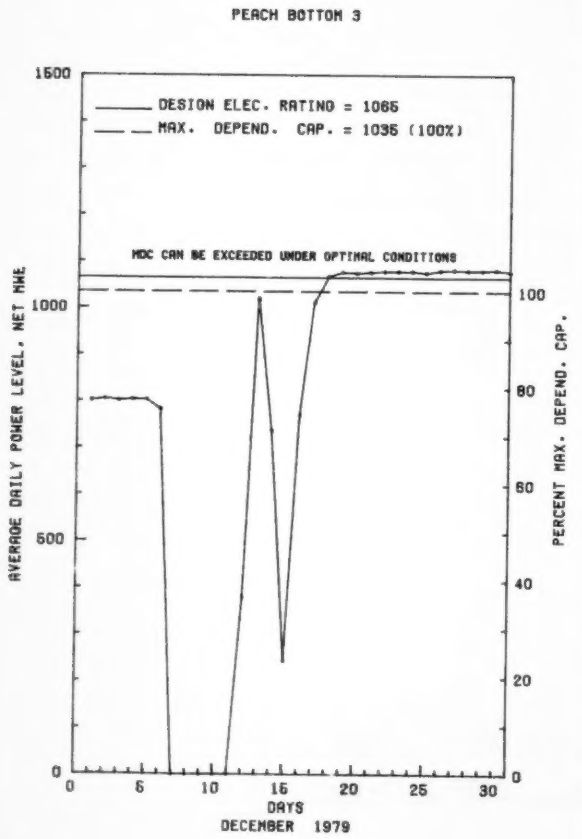
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-033/ 03L	10/22/79	11/21/79	30 Day LER	While in a refueling outage with two RHR pumps blocked for modification work a third RHR pump tripped on overcurrent Tech Spec 3.5.F.4 applied
79-034/ 03L	11/01/79	11/28/79	30 Day LER	Standby liquid control pump relief valve (1) operated below Tech Spec limit
79-035/ 03L	11/02/79	12/03/79	30 Day LER	Tip System shear valves installed backwards
79-036/ 03L	11/05/79	12/05/79	30 Day LER	Leak rate surveillance tests showed leak rates exceeding the Tech Spec limits for drywell sump isolation valve N2 injection check valve 2 HPCI and 2 RCIC valves and 7 MSIVs
79-037/	11/13/79	12/13/79	30 Day LER	The sequence mode of the rod sequence control system was found to be inoperable prior to startup
79-038/ 01T	11/21/79	12/06/79	24 Hours	Two MSIVs closed faster than allowed by TS during surveillance
79-039/ 01P	11/30/79	12/04/79	24 Hours	In response to Bulletin 79-02 four seismic supports associated with the 1" CRD insert and withdraw piping were found to be not seismically qualified

REVIEWED BY OMPA LOEB: John J. **DATE:** 2-11-80

REVIEWED BY NRR DOR: Dick Clark **DATE:** 02-13-80

PEACH BOTTOM 3

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-278
2. Utility Contact: W. M. Alden (215) 841-5022
3. Licensed Thermal Power (MWt): 3293
4. Nameplate Rating (Gross MWe): 1280 x 0.9 = 1152
5. Design Electrical Rating (Net MWe): 1065
6. Maximum Dependable Capacity (Gross MWe): 1098
7. Maximum Dependable Capacity (Net MWe): 1035
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	8,760.0	44,040.0
12. Hours Reactor Was Critical	616.0	6,736.0	34,846.0
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	588.6	6,501.9	33,931.8
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,779,098	19,345,018	94,219,154
17. Gross Electrical Energy (MWH)	595,390	6,357,750	30,585,860
18. Net Electrical Energy (MWH)	575,852	6,101,657	29,344,800
19. Unit Service Factor	79.1	74.2	77.0
20. Unit Availability Factor	79.1	74.2	77.0
21. Unit Capacity Factor (MDC Net)	74.8	67.3	64.4
22. Unit Capacity Factor (DER Net)	72.7	65.4	62.6
23. Unit Forced Outage Rate	3.0	3.6	6.9

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
20	12/07/79	S	137.5	B	1		CD	VALVOP	SHUTDOWN FOR REPAIR OF 'A' MAIN STEAM LINE ISOLATION VALVE.
21	12/14/79	F	17.9	A	2		EB	INSTRU	MANUAL SCRAM AFTER TEMPORARY LOSS OF POWER TO INSTRUMENTATION.

SUMMARY

OPERATION WAS AT NEAR FULL POWER DURING THE LAST HALF OF THE REPORT PERIOD. ON 12/07/79, THE UNIT WAS SHUTDOWN TO REPAIR THE "A" MAIN STEAM LINE ISOLATION VALVE. A MANUAL SCRAM OCCURRED ON 12/14/79 FOLLOWING TEMPORARY LOSS OF POWER TO INSTRUMENTATION.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

PILGRIM

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Massachusetts/Plymouth 4 miles SE of Plymouth
2.	Electric Reliability Council	NPCC
3.	Type of Reactor	BWR
4.	Licenses Power Level (MWT)	1998
5.	Design Electrical Rating (MWE Net)	655
6.	Date of Initial Criticality	June 16, 1972
7.	Date of Commercial Operation	December 1972
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Cape Cod Bay

Utility & Contractor Information

10.	Licensee	Boston Edison Company
11.	Corporate Address	800 Boylston Street Boston, Massachusetts 02199
12.	Corporate Contact	A. Corey, Vice President
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	GE
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	IE Senior Inspector	E. McCabe
19.	IE Principal Inspector	W. Baunack
20.	Licensing Project Manager	J. Hannon
21.	Docket Number	50-293
22.	License Number/Date of Issue	DPR-35, 6/8/72
23.	Public Document Room	Plymouth Public Library North Street Plymouth, Massachusetts 02360

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

None

FACILITY ITEMS (PLANS AND PROCEDURES):

None

MANAGERIAL ITEMS:

None.

PLANT STATUS:

+ The facility was shutdown on January 5 for a refueling outage scheduled to be completed in mid-April.

+ LAST IE SITE INSPECTION DATE: 01/8-11/80

+ INSPECTION REPORT NO: 80-01

SUMMARY

INSPECTION SUMMARY

+ Dec 3-7 (79-20): Routine, unannounced inspection by two region-based inspectors (68 hours onsite) of plant operations. A facility tour was conducted. Noncompliances: One (failure to report in accordance with Technical Specification requirements).

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

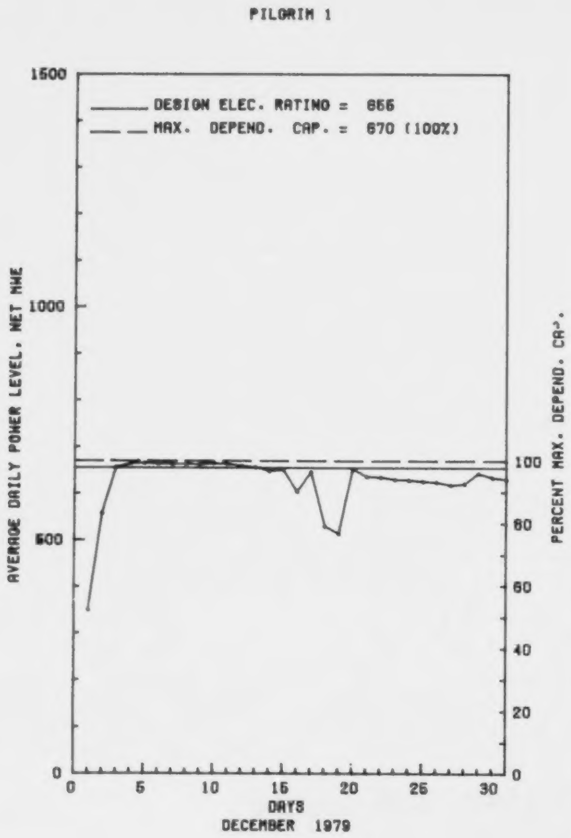
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-002/ OR	12/11/79	12/28/79	Special	Accidental lifting of spent fuel element while loading new fuel into spent fuel pit
79-040/ 03L	10/26/79	11/26/79	30 Day LER	RCS leak detection air sampler inoperable
79-041/ 04T	11/19/79	12/11/79	14 Day LER	Composite sample greater than 10 times lower limit of detection
79-042/ 03L	11/01/79	11/30/79	30 Day LER	Containment isolation valve failures
79-043/ 03L	11/08/79	12/06/79	30 Day LER	Drywell unidentified leakage in excess of Technical Specification limits
79-044/ 03L	11/09/79	12/06/79	30 Day LER	Reactor coolant leak detection air sampling system failure
79-045/ 03L	11/14/79	12/11/79	30 Day LER	Failure to maintain drywell to torus differential pressure
79-046/ 03L	08/12/79	12/14/79	30 Day LER	Station organization change

REVIEWED BY OMPA LOEB: John J. Riggs DATE: 2-11-80

REVIEWED BY NRR DOR: John N Hannon DATE: 2-13-80

PILGRIM

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-293
2. Utility Contact: G. G. Whitney (617) 746-7900
3. Licensed Thermal Power (MWt): 1998
4. Nameplate Rating (Gross MWe): 780 x 0.87 = 678
5. Design Electrical Rating (Net MWe): 655
6. Maximum Dependable Capacity (Gross MWe): 690
7. Maximum Dependable Capacity (Net MWe): 670
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	8,760.0	61,896.0
12. Hours Reactor Was Critical	744.0	7,933.2	44,980.9
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	744.0	7,829.9	43,552.5
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,404,960	14,649,312	73,091,112
17. Gross Electrical Energy (MWH)	483,780	5,034,363	24,164,834
18. Net Electrical Energy (MWH)	464,639	4,844,559	23,206,123
19. Unit Service Factor	100.0	89.4	70.4
20. Unit Availability Factor	100.0	89.4	70.4
21. Unit Capacity Factor (MDC Net)	93.2	82.5	56.0
22. Unit Capacity Factor (DER Net)	95.3	84.4	57.2
23. Unit Forced Outage Rate	.0	10.2	10.6

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUELING - 01/05/80 THRU 04/07/80.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
19	12/01/79	S	.0	B	5				CONDENSER BACKWASH AND LEAK CHECK.
20	12/18/79	F	.0	H	5				'B' RECIRC. M. G. SET TRIPPED DUE TO FAULT IN CABLE.

SUMMARY

OPERATION WAS ROUTINE DURING THE REPORT PERIOD, EXCEPT FOR A "B" RECIRCULATING MOTOR GENERATOR SET TRIP ON 12/18/79 DUE TO A CABLE FAULT. LOAD WAS REDUCED ON 12/01/79 TO BACKWASH THE MAIN CONDENSER AND TO CHECK FOR AIR IN-LEAKAGE, AND ON 12/16/79 FOR CONTROL ROD ADJUSTMENT. THE UNIT BEGAN END OF CYCLE COASTDOWN ON 12/11/79.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

POINT BEACH 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Wisconsin/Manitowoc 15 miles N of Manitowoc
2.	Electric Reliability Council	
3.	Type of Reactor	PWR
4.	Power Level (MWT)	1518
5.	Design Electrical Rating (MWE Net)	497
6.	Date of Initial Criticality	November 2, 1970
7.	Date of Commercial Operation	December 21, 1970
8.	Condenser Cooling Method	Straight Through
9.	Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10.	Licensee	Wisconsin Electric Power Co.
11.	Corporate Address	231 West Michigan Milwaukee, Wisconsin
12.	Corporate Contact	S. Burstein
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Senior Inspector	R. F. Warnick
19.	IE Resident Inspector	B.L. Jorgnesen
20.	Licensing Project Manager	C. M. Trammell
21.	Docket Number	50-266
22.	License Number/Date of Issue	DPR-24 issued 10/05/70
23.	Public Document Room	Document Department, Library University of Wisconsin-Stevens Point Stevens Point, Wisconsin 54481 Manitowoc Public Library Manitowoc, Wisconsin (Temporary LPDR for spent fuel pool capacity increase amendment only

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

+ The plant was shutdown due to tube leak in steam generator "B". Licensee performed eddy current testing and tube plugging in both steam generators and returned to power on December 23, 1979.

+LAST IE SITE INSPECTION DATE: October 23-24, 1979

+ INSPECTION REPORT NO: 79-15

SUMMARY

INSPECTION SUMMARY

+ Inspection on October 23-24, (79-15): Routine, unannounced inspection of confirmatory measurements and radiological environmental protection for Unit 1, including licensee program for quality control of analytical measurements; comparison of analytical results of samples collected previously; collection of air particulate, charcoal adsorber, waste gas and liquid samples for future comparison; management controls; implementation of radiological environmental monitoring program. The inspection involved 12 inspector hours onsite by two NRC inspectors. No apparent items of noncompliance or deviations were identified during this inspection.

REPORTS RECEIVED FROM LICENSEE

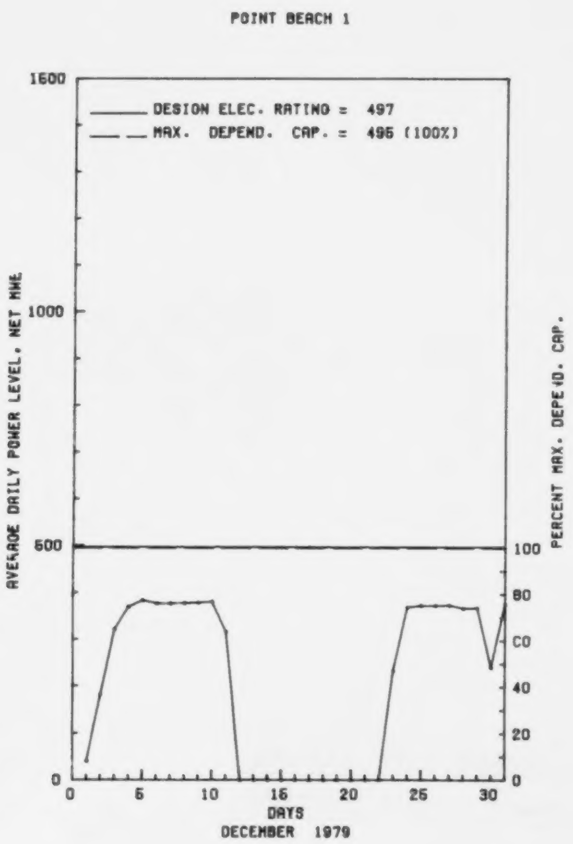
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-20/01T-0	11/15/79	11/27/79	14 Day LER	Anchor in "A" SG auxiliary feedwater line contained construction deficiency.

REVIEWED BY OMPA LOEB: T.C. Cintula DATE: 2-11-80

REVIEWED BY NRR DOR: CM Trammell DATE: 2-13-80

POINT BEACH 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

ITEM 21 CALCULATED BY USING A WEIGHTED AVERAGE

1. Docket Number: 50-266
2. Utility Contact: C.W. FAY (414) 277-2811
3. Licensed Thermal Power (MWt): 1518
4. Nameplate Rating (Gross MWe): 582 x 0.9 = 524
5. Design Electrical Rating (Net MWe): 497
6. Maximum Dependable Capacity (Gross MWe): 519
7. Maximum Dependable Capacity (Net MWe): 495
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): 390
10. Reasons for Restrictions, If Any: CORROSION INHIB. OF STM. GEN. TUBES.

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	8,760.0	80,232.0
12. Hours Reactor Was Critical	477.5	6,628.6	66,516.5
13. Reactor Reserve Shutdown Hours	7.3	142.6	582.2
14. Hours Generator On-Line	465.6	6,457.0	64,394.0
15. Unit Reserve Shutdown Hours	11.8	219.0	602.4
16. Gross Thermal Energy (MWH)	507,068	9,386,180	90,752,795
17. Gross Electrical Energy (MWH)	166,920	3,206,110	30,633,750
18. Net Electrical Energy (MWH)	156,484	3,055,424	29,190,178
19. Unit Service Factor	62.6	73.7	80.3
20. Unit Availability Factor	64.2	76.2	81.0
21. Unit Capacity Factor (MDC Net)	42.5	70.5	74.8
22. Unit Capacity Factor (DER Net)	42.3	70.2	73.2
23. Unit Forced Outage Rate	36.6	9.9	3.7

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): STM GEN LEAK TEST AND EDDY INSPEC - 2/29/80.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
7	10/05/79	S	9.2	C	4				REFUELING OUTAGE CONTINUED.
8	12/11/79	F	269.2	A	1	79-021	CB	HTEXCH	A SHUTDOWN OF THE UNIT COMMENCED AT 1713 HOURS ON 12/10/79 FOLLOWING CONFIRMATION OF PRIMARY-TO-SECONDARY LEAKAGE OF 250 TO 260 GALLONS PER DAY. LEAKING TUBE WAS PLUGGED AND THE PLUGS WERE WELD REPAIRED. EDDY CURRENT INSPECTION WAS ALSO PERFORMED AND ALL TUBES WITH INDICATIONS WERE PLUGGED.
9	12/30/79	F	.0	B	5		HA	TURBIN	A LOAD SWING FROM FULL POWER (390 MWE NET) TO 100 MWE AND BACK TO FULL POWER WAS PERFORMED DURING TURBINE VIBRATION ANALYSIS.

SUMMARY

THE UNIT RETURNED TO POWER FROM A REFUELING OUTAGE ON 12/01/79, AND OPERATED AT REDUCED POWER UNTIL 12/11/79 WHEN IT WAS SHUTDOWN TO CORRECT PRIMARY-TO-SECONDARY LEAKAGE. A NO-POWER TRIP OCCURRED DURING STARTUP ON 12/23/79. ON 12/30/79, LOAD WAS BRIEFLY REDUCED DURING TURBINE VIBRATION ANALYSIS.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

POINT BEACH 2

FACILITY DATA

Facility Description

1. Location (state/county) (nearest population center): Wisconsin/Manitowoc 15 miles N of Manitowoc
2. Electric Reliability Council:
3. Type of Reactor: PWR
4. Power Level (MWT): 1518
5. Design Electrical Rating (MWE Net): 497
6. Date of Initial Criticality: May 30, 1972
7. Date of Commercial Operation: October 1, 1972
8. Condenser Cooling Method: Straight Through
9. Condenser Cooling Water Source: Lake Michigan

Utility & Contractor Information

10. Licensee: Wisconsin Electric Power Co.
11. Corporate Address: 231 West Michigan Milwaukee, Wisconsin
12. Corporate Contact: S. Burstein
13. Architect/Engineer: Bechtel
14. Nuclear Steam Supply System: Westinghouse
15. Constructor: Bechtel

Regulatory Information

16. IE Region responsible: III
17. IE Branch Chief: R. Heishman
18. IE Senior Inspector: R. F. Warnick
19. IE Resident Inspector: B.L. Jorgnesen
20. Licensing Project Manager: C. M. Trammell
21. Docket Number: 50-301
22. License Number/Date of Issue: DPR-27 issued 11/16/71
23. Public Document Room:

Document Department, Library
University of Wisconsin-Stevens Point
Stevens Point, Wisconsin 54481

Manitowoc Public Library
Manitowoc, Wisconsin
(Temporary LPDR for
spent fuel pool capacity
increase amendment only

P-12

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

The plant operated routinely at power during the month.

*LAST IE SITE INSPECTION DATE: October 23-24, 1979

* INSPECTION REPORT NO: 79-17

SUMMARY

INSPECTION SUMMARY

* Inspection on October 23-24, (79-17): Routine, unannounced inspection of confirmatory measurements and radiological environmental protection for Unit 2, including licensee program for quality control of analytical measurements; comparison of analytical results of samples collected previously; collection of air particulate, charcoal adsorber, waste gas and liquid samples for future comparison; management controls; implementation of radiological environmental monitoring program. The inspection involved 12 inspector hours onsite by two NRC inspectors. No apparent items of noncompliance or deviations were identified during this inspection.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

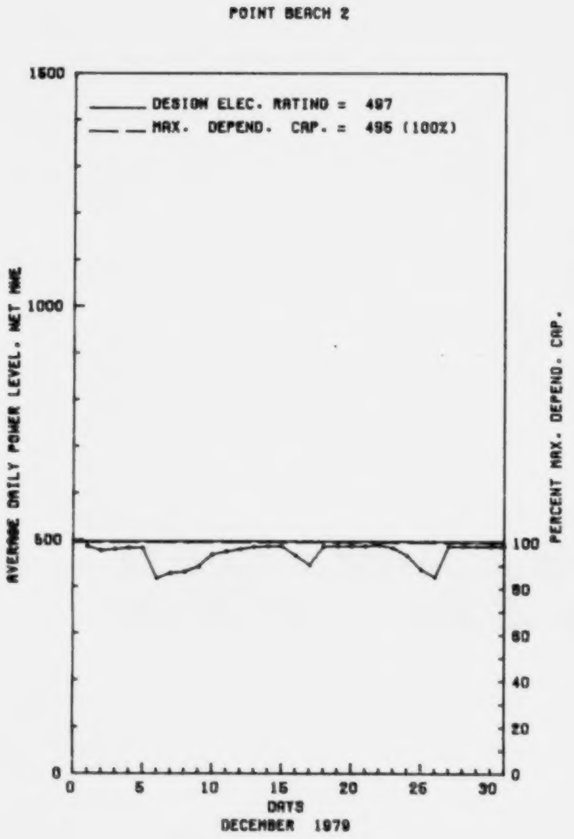
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
	NONE			

REVIEWED BY OMPA LOEB: T. C. Cintula DATE: 2/11/80

REVIEWED BY NRR DOR: CM Trammell DATE: 2-13-80

POINT BEACH 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

ITEM 21 CALCULATED
BY USING A WEIGHTED
AVERAGE

1. Docket Number: 50-301
2. Utility Contact: C.W. FAY (414) 277-2811
3. Licensed Thermal Power (MWt): 1518
4. Nameplate Rating (Gross MWe): 582 x 0.9 = 524
5. Design Electrical Rating (Net MWe): 497
6. Maximum Dependable Capacity (Gross MWe): 519
7. Maximum Dependable Capacity (Net MWe): 495
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	8,760.0	65,017.0
12. Hours Reactor Was Critical	744.0	7,807.8	58,896.9
13. Reactor Reserve Shutdown Hours	.0	16.5	166.1
14. Hours Generator On-Line	744.0	7,732.2	57,727.0
15. Unit Reserve Shutdown Hours	.0	18.6	106.4
16. Gross Thermal Energy (MWH)	1,095,273	11,367,319	77,652,536
17. Gross Electrical Energy (MWH)	366,860	3,886,230	26,361,340
18. Net Electrical Energy (MWH)	350,507	3,707,450	25,075,127
19. Unit Service Factor	100.0	88.3	88.8
20. Unit Availability Factor	100.0	88.5	89.0
21. Unit Capacity Factor (MDC Net)	95.2	85.5	78.7
22. Unit Capacity Factor (DER Net)	94.8	85.2	77.6
23. Unit Forced Outage Rate	.0	.1	1.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
REFUELING - 03/28/80 - 5 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	NONE								

SUMMARY

OPERATION WAS ROUTINE DURING THE REPORT PERIOD, WITH BRIEF LOAD REDUCTIONS FOR SLUDGE LANCING, TO PLUG TUBE LEAKS IN A CONDENSER WATERBOX, AND ON REQUEST FROM POWER SUPPLY.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

PRAIRIE ISLAND 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1. Location (state/county) (nearest population center)	Minnesota/Goodhue 28 miles SE of Minneapolis
2. Electric Reliability Council	MARCA
3. Type of Reactor	PWR
4. Power Level (MWT)	1650
5. Design Electrical Rating (MWE Net)	530
6. Date of Initial Criticality	December 1, 1973
7. Date of Commercial Operation	December 16, 1973
8. Condenser Cooling Method	Cooling Towers
9. Condenser Cooling Water Source	Mississippi River

Utility & Contractor Information

10. Licensee	Northern States Power Company
11. Corporate Address	414 Nicollet Mall - 8th Floor Minneapolis, Minnesota 55401
12. Corporate Contact	G. H. Neils (Oper.) L. O. Mayer (Lic.)
13. Architect/Engineer	Pioneer Services & Eng. Co.
14. Nuclear Steam Supply System	Westinghouse
15. Constructor	Northern States Power Co.

Regulatory Information

16. IE Region responsible	III
17. IE Branch Chief	R. Heishman
18. IE Senior Inspector	R. F. Warnick
19. IE Resident Inspector	C. D. Feierabend 612-388-8209
20. Licensing Project Manager	M. Grotenhuis
21. Docket Number	50-282
22. License Number/Date of Issue	DPR-42 issued 8/9/73
23. Public Document Room	Environmental Conservation Library Minneapolis Public Library 300 Nicollet Mall Minneapolis, Minnesota 55401

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

The plant was operating at the end of the month.

+LAST IE SITE INSPECTION DATE: October 10-12, 1979

+ INSPECTION REPORT NO: 79-24

SUMMARY

INSPECTION SUMMARY

+ Inspection on October 10-12, (79-24): Routine, unannounced inspection of the radiation protection program, including: qualifications; audits; training; radiation protection procedures; instruments and equipment; exposure control; posting, labeling, and control; surveys; notifications and reports; IE Bulletin 79-19; IE Circular 79-09, IE Information Notice 79-08; review of licensee actions on Radiation Area Monitors identified in Reports 78-05 and 78-07; handling of the noncompliance item from Report No. 79-03; contract radiation technician qualifications; and a review of licensee actions during the recent tube rupture (October 2, 1979). The inspection involved 55 inspector hours onsite by two NRC inspectors. No items of noncompliance or deviations were identified.

REPORTS RECEIVED FROM LICENSEE

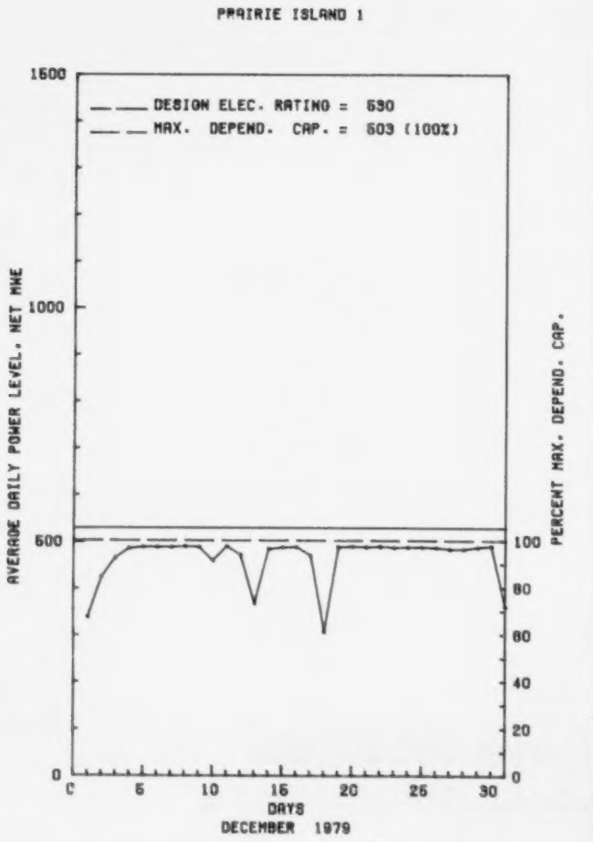
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
NONE				

REVIEWED BY OMPA LOEB: T.C. CINTULA DATE: 2-11-80

REVIEWED BY NRR DOR: M Grotenhuis DATE: 2-13-80

PRAIRIE ISLAND 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-282
2. Utility Contact: A. A. Hunstad (612) 388-4767
3. Licensed Thermal Power (MWt): 1650
4. Nameplate Rating (Gross MWe): 659 x 0.9 = 593
5. Design Electrical Rating (Net MWe): 530
6. Maximum Dependable Capacity (Gross MWe): 534
7. Maximum Dependable Capacity (Net MWe): 503
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	8,760.0	52,968.0
12. Hours Reactor Was Critical	744.0	6,519.3	41,208.4
13. Reactor Reserve Shutdown Hours	.0	1,076.8	5,531.9
14. Hours Generator On-Line	734.1	6,406.3	40,123.6
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,171,117	9,798,412	62,078,994
17. Gross Electrical Energy (MWH)	369,060	3,122,780	20,153,670
18. Net Electrical Energy (MWH)	345,989	2,910,820	18,839,252
19. Unit Service Factor	98.7	73.1	75.8
20. Unit Availability Factor	98.7	73.1	75.8
21. Unit Capacity Factor (MDC Net)	92.5	66.1	70.7
22. Unit Capacity Factor (DER Net)	87.7	62.7	67.1
23. Unit Forced Outage Rate	.0	19.2	13.6

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
REFUELING, SUMMER 1980, 5 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	12/18/79	S	6.7	B	1		CH	PIPEXX	SHUTDOWN TO REPAIR A LEAK IN A FEEDWATER SAMPLE TAP.
	12/31/79	S	3.2	B	1		CA	VALVEX	SHUTDOWN TO REPAIR A PACKING LEAK ON AN RTD BYPASS VENT VALVE.

SUMMARY

OPERATION WAS ROUTINE DURING THE REPORT PERIOD, EXCEPT FOR OUTAGES ON 12/18/79 TO REPAIR A LEAKING FEEDWATER SAMPLE TAP, AND ON 12/31/79 TO REPAIR A PACKING LEAK ON A RESISTANCE TEMPERATURE DETECTOR BYPASS VENT VALVE

LEGEND

[1]Type:
F--Forced
S--Scheduled

[2]Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3]Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4]Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5]Exhibit H:
Same source.

PRAIRIE ISLAND 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Minnesota/Goodhue 28 miles SE of Minneapolis
2.	Electric Reliability Council	MARCA
3.	Type of Reactor	PWR
4.	Power Level (MWT)	1650
5.	Design Electrical Rating (MWE Net)	530
6.	Date of Initial Criticality	December 17, 1974
7.	Date of Commercial Operation	December 21, 1974
8.	Condenser Cooling Method	Cooling Towers
9.	Condenser Cooling Water Source	Mississippi River

Utility & Contractor Information

10.	Licensee	Northern States Power Company
11.	Corporate Address	414 Nicollet Mall - 8th Floor Minneapolis, Minnesota 55401
12.	Corporate Contact	G. H. Neils (Oper.) L. O. Mayer (Lic.)
13.	Architect/Engineer	Pioneer Services & Eng. Co.
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Northern States Power Co.

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Senior Inspector	R. F. Warnick
19.	IE Resident Inspector	C. D. Feierabend 612-388-8209
20.	Licensing Project Manager	M. Grotenhuis
21.	Docket Number	50-306
22.	License Number/Date of Issue	DPR-60 issued 10/29/74
23.	Public Document Room	Environmental Conservation Library Minneapolis Public Library 300 Nicollet Mall Minneapolis, Minnesota 55401

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

* The plant was operating routinely at the end of the month. Refueling outage is scheduled to start January 2, 1980.

*LAST IE SITE INSPECTION DATE: October 10-12, 1979

* INSPECTION REPORT NO: 79-20

SUMMARY

INSPECTION SUMMARY

* Inspection on October 10-12, (79-20): Routine, unannounced inspection of the radiation protection program, including: qualifications; audits; training; radiation protection procedures; instruments and equipment; exposure control; posting, labeling, and control; surveys; notifications and reports; IE Bulletin 79-19; IE Circular 79-09, IE Information Notice 79-08; review of licensee actions on Radiation Area Monitors identified in Reports 78-05 and 78-07; handling of the noncompliance item from Report No. 79-03; contract radiation technician qualifications; and a review of licensee actions during the recent tube rupture (October 2, 1979). The inspection involved 55 inspector hours onsite by two NRC inspectors. No items of noncompliance or deviations were identified.

REPORTS RECEIVED FROM LICENSEE

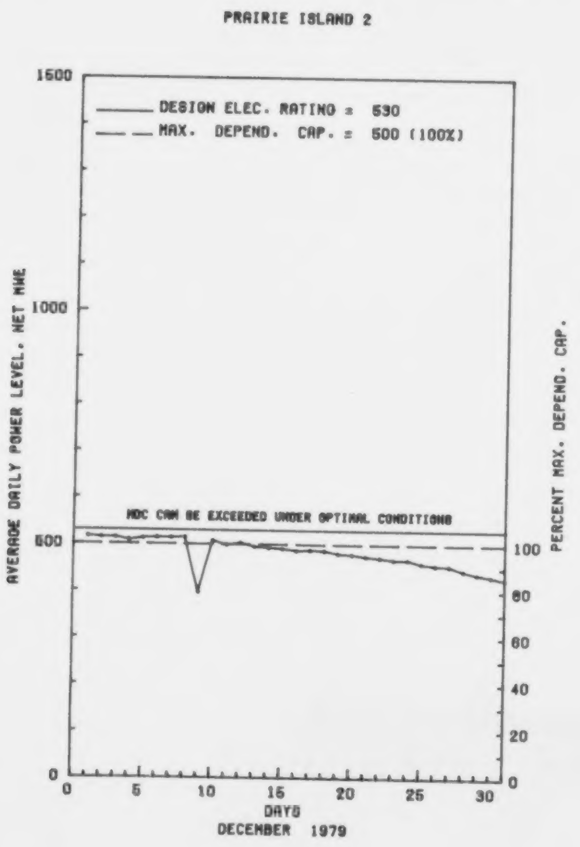
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
NONE				

REVIEWED BY OMPA LOEB: T.C. CINTULA DATE: 20110 80

REVIEWED BY NRR DOR: M Grotenhuis DATE: 2-13-80

PRAIRIE ISLAND 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-306
2. Utility Contact: A. A. Hunstad (612) 388-4767
3. Licensed Thermal Power (MWt): 1650
4. Nameplate Rating (Gross MWe): 659 x 0.9 = 593
5. Design Electrical Rating (Net MWe): 530
6. Maximum Dependable Capacity (Gross MWe): 531
7. Maximum Dependable Capacity (Net MWe): 500
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	8,760.0	44,086.0
12. Hours Reactor Was Critical	744.0	8,685.4	39,129.0
13. Reactor Reserve Shutdown Hours	.0	27.3	1,406.0
14. Hours Generator On-Line	744.0	8,665.4	38,401.0
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,139,812	13,653,218	59,672,503
17. Gross Electrical Energy (MWH)	381,180	4,466,390	19,054,130
18. Net Electrical Energy (MWH)	357,882	4,193,044	17,838,632
19. Unit Service Factor	100.0	98.9	87.1
20. Unit Availability Factor	100.0	98.9	87.1
21. Unit Capacity Factor (MDC Net)	96.2	95.7	80.9
22. Unit Capacity Factor (DER Net)	90.8	90.3	76.3
23. Unit Forced Outage Rate	.0	.6	4.8

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUELING, JANUARY 2, 1980, 5 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	NONE								

SUMMARY

OPERATION WAS ROUTINE IN AN END OF CYCLE COASTDOWN, WITH NO FURTHER REDUCTIONS OR OUTAGES REPORTED.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

QUAD CITIES 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Illinois/Rock Island 20 miles NE of Moline
2.	Electric Reliability Council	Main
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2511
5.	Design Electrical Rating (MWE Net)	789
6.	Date of Initial Criticality	October 18, 1971
7.	Date of Commercial Operation	February 18, 1973
8.	Condenser Cooling Method	Spray Canal
9.	Condenser Cooling Water Source	Mississippi River

Utility & Contractor Information

10.	Licensee	Commonwealth Edison Co.
11.	Corporate Address	P. O. Box 767 Chicago, Illinois 60690
12.	Corporate Contact	D. L. Peoples
13.	Architect/Engineer	Sargent and Lundy
14.	Nuclear Steam Supply System	GE
15.	Constructor	United Engineers & Constructors, Inc.

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Senior Inspector	R. Spessard
19.	IE Resident Inspector	N. J. Chrissotimos
20.	Licensing Project Manager	R. B. Bevan
21.	Docket Number	50-254
22.	License Number/Date of Issue	DPR-29 issued 10/1/71
23.	Public Document Room	Moline Public Library 504 17th Street Moline, Illinois 61265

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

FACILITY ITEMS(PLANS AND PROCEDURES):

+ The licensee is constructing a temporary building for LSA material so as to increase storage capability until shipping problems are resolved. (79-22)

PLANT STATUS

Unit operating routinely.

+LAST IE SITE INSPECTION DATE: October 29 through November 2, 1979

+ INSPECTION REPORT NO: 79-23

SUMMARY

INSPECTION SUMMARY

+ Inspection on October 3 through November 2, (79-22): Routine, unannounced resident inspection of maintenance, plant operations, physical protection-security organization and access control (identification, authorization, badging, search and escorting), review and followup on licensee event reports, information meetings with local officials, GSEP review, and refueling activities. The inspection involved 68 inspector hours onsite by one NRC inspector. Of the nine areas inspected, no apparent items of noncompliance were identified.

+ Inspection on October 29 through November 2, (79-23): Routine, announced inspection of emergency planning, including an examination of the licensee's audit of their Emergency Plan, coordination with offsite agencies, facilities and equipment, training, drills and meteorological and effluent monitoring systems; radiological environmental program; and confirmatory measurements program. The inspection involved 49 inspector hours onsite by two NRC inspectors. For the eight areas inspected, no apparent items of noncompliance or deviations were noted.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

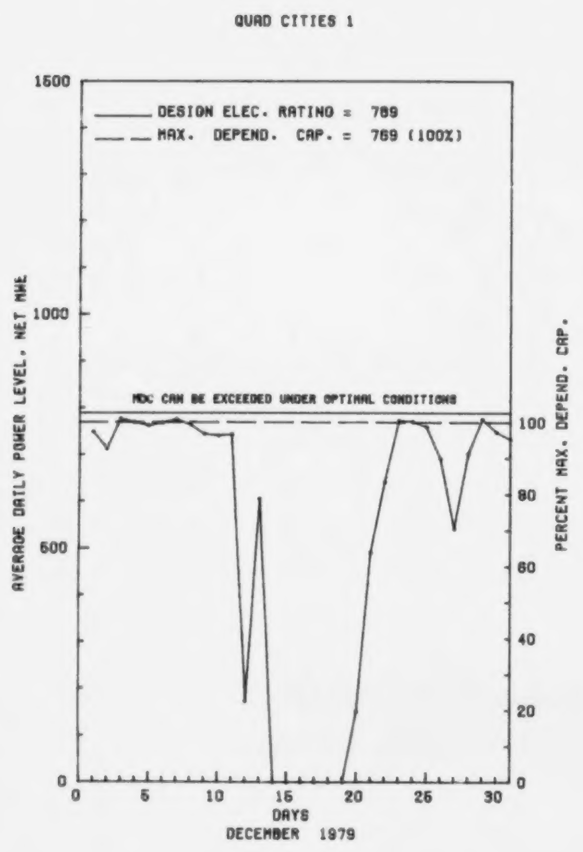
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-34/ 03L-0	10/30/79	11/22/79	30 Day LER	Suppression pool level below Technical Specification limits.
79-35/ 03L-0	11/09/79	11/30/79	30 Day LER	Pipe restraint declared inoperable as a result of IEB No. 79-14 inspection.
79-36/ 03L-0	11/15/79	12/07/79	30 Day LER	SBGT charcoal filter freon penetration in excess of TS limit.
79-38/ 01T-0	11/28/79	12/10/79	14 Day LER	ECCS Level setpoints nonconservative due to inaccurate temp compensation.

REVIEWED BY OMPA LOEB: DATE: 2/11/80

REVIEWED BY NRR DOR: Dick Clark DATE: 02-14-80

QUAD CITIES 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-254
2. Utility Contact: D. Hannum (309) 654-2241
3. Licensed Thermal Power (MWt): 2511
4. Nameplate Rating (Gross MWe): 920 x 0.9 = 828
5. Design Electrical Rating (Net MWe): 789
6. Maximum Dependable Capacity (Gross MWe): 813
7. Maximum Dependable Capacity (Net MWe): 769
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	8,760.0	66,960.0
12. Hours Reactor Was Critical	681.7	7,381.2	54,714.1
13. Reactor Reserve Shutdown Hours	.0	92.3	3,421.9
14. Hours Generator On-Line	578.8	7,098.0	52,040.9
15. Unit Reserve Shutdown Hours	.0	19.8	909.2
16. Gross Thermal Energy (MWH)	1,287,015	15,736,105	104,543,981
17. Gross Electrical Energy (MWH)	424,452	5,050,559	33,623,054
18. Net Electrical Energy (MWH)	403,160	4,782,963	31,415,536
19. Unit Service Factor	77.8	81.0	77.7
20. Unit Availability Factor	77.8	81.3	79.1
21. Unit Capacity Factor (MDC Net)	70.5	71.0	61.0
22. Unit Capacity Factor (DER Net)	68.7	69.2	59.5
23. Unit Forced Outage Rate	2.1	5.4	7.8

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
26	12/11/79	F	12.6	A	3		EB	CKTBRK	REACTOR SCRAM DUE TO BLOWN POTENTIAL TRANSFORMER FUSES.
27	12/14/79	S	152.6	B	1				MAINTENANCE OUTAGE FOR 125/250 VOLT BATTERY TEST AND NUREG 0578 MODIFICATION REQUIREMENTS.

SUMMARY

OPERATION WAS ROUTINE DURING THE REPORT PERIOD, WITH A REACTOR SCRAM DUE TO BLOWN POTENTIAL TRANSFORMER FUSES ON 12/11/79. THE UNIT WAS SHUTDOWN FROM 12/14/79 THROUGH 12/19/79 FOR BATTERY TESTING AND CHECKING RELIEF VALVE POSITION INDICATION MANUAL RESET OF CONTAINMENT SOLUTION, AND CORE DAMAGE ASSESSMENT RADIATION MONITORS.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

QUAD CITIES 2

FACILITY DATA

Facility Description

1. Location (state/county) (nearest population center)	Illinois/Rock Island 20 miles NE of Moline
2. Electric Reliability Council	Main
3. Type of Reactor	BWR
4. Power Level (MWT)	2511
5. Design Electrical Rating (MWE Net)	789
6. Date of Initial Criticality	April 26, 1972
7. Date of Commercial Operation	March 10, 1973
8. Condenser Cooling Method	Spray Canal
9. Condenser Cooling Water Source	Mississippi River

Utility & Contractor Information

10. Licensee	Commonwealth Edison Co.
11. Corporate Address	P. O. Box 767 Chicago, Illinois 60690
12. Corporate Contact	D. L. Peoples
13. Architect/Engineer	Sargent & Lundy
14. Nuclear Steam Supply System	GE
15. Constructor	United Engineers & Constructors, Inc.

Regulatory Information

16. IE Region responsible	III
17. IE Branch Chief	R. Heishman
18. IE Senior Inspector	R. Spessard
19. IE Resident Inspector	N. J. Chrissotimos
20. Licensing Project Manager	R. B. Bevan
21. Docket Number	50-265
22. License Number/Date of Issue	DPR-30 issued 3/21/72
23. Public Document Room	Moline Public Library 504 17th Street Moline, Illinois 61265

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

FACILITY ITEMS(PLANS AND PROCEDURES):

- The licensee is constructing a temporary building for LSA material so as to increase storage capability until shipping problems are resolved. (79-19)
- Unit 2 refuel outage has been rescheduled and is expected to begin November 25, 1979 and last approximately 12 weeks. (79-19)

PLANT STATUS

The unit was shut down on November 25, 1979 to begin a scheduled 13-week refueling and maintenance outage.

+LAST IE SITE INSPECTION DATE: October 29 through Novembe: 2, 1979

+ INSPECTION REPORT NO: 79-20

SUMMARY

INSPECTION SUMMARY

- Inspection on October 3 through November 2, (79-19): Routine, unannounced resident inspection of maintenance, plant operations, physical protection-security organization and access control (identification, authorization, badging, search and escorting), review and followup on licensee event reports, information meetings with local officials, GSEP review, and refueling activities. The inspection involved 68 inspector hours onsite by one NRC inspector. Of the nine areas inspected, no apparent items of noncompliance were identified.
- Inspection on October 29 through November 2, (79-20): Routine, announced inspection of emergency planning, including an examination of the licensee's audit of their Emergency Plan, coordination with offsite agencies, facilities and equipment, training, drills and meteorological and effluent monitoring systems; radiological environmental program; and confirmatory measurements program. The inspection involved 49 inspector hours onsite by two NRC inspectors. For the eight areas inspected, no apparent items of noncompliance or deviations were noted.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

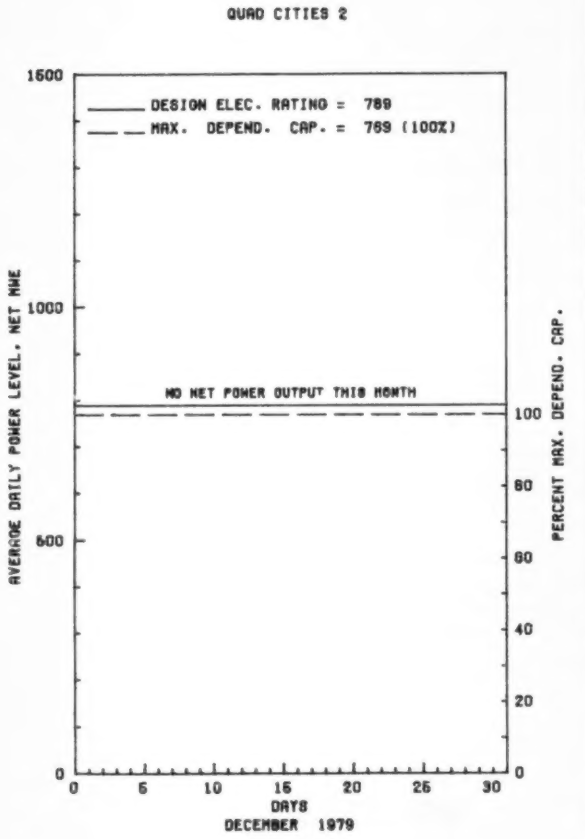
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT

REVIEWED BY OMPA LOEB: DATE: 2/11/80

REVIEWED BY NRR DOR: Dick Clark DATE: 02-14-80

QUAD CITIES 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-265
2. Utility Contact: D. Hannum (309) 654-2241
3. Licensed Thermal Power (MWt): 2511
4. Nameplate Rating (Gross MWe): 920 x 0.9 = 828
5. Design Electrical Rating (Net MWe): 789
6. Maximum Dependable Capacity (Gross MWe): 813
7. Maximum Dependable Capacity (Net MWe): 769
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	8,760.0	66,070.0
12. Hours Reactor Was Critical	.0	7,752.5	53,128.9
13. Reactor Reserve Shutdown Hours	.0	.0	2,985.8
14. Hours Generator On-Line	.0	7,688.9	50,791.8
15. Unit Reserve Shutdown Hours	.0	.0	702.9
16. Gross Thermal Energy (MWH)	0	14,273,985	103,816,109
17. Gross Electrical Energy (MWH)	15	4,374,673	32,863,050
18. Net Electrical Energy (MWH)	-12,405	3,981,065	30,742,525
19. Unit Service Factor	.0	87.8	76.9
20. Unit Availability Factor	.0	87.8	77.9
21. Unit Capacity Factor (MDC Net)	.0	59.1	60.5
22. Unit Capacity Factor (DER Net)	.0	57.6	59.0
23. Unit Forced Outage Rate	.0	.9	9.3

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 02/29/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
13	11/25/79	S	744.0	C	4				UNIT TWO CYCLE 4 REFUELING OUTAGE CONTINUES.

SUMMARY

THE UNIT REMAINED SHUTDOWN THROUGHOUT THE REPORT PERIOD FOR REFUELING.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

RANCHO SECO

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	California/Sacramento 25 miles SE of Sacramento
2.	Electric Reliability Council	WSCC
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2772
5.	Design Electrical Rating (MWE Net)	918
6.	Date of Initial Criticality	September 16, 1974
7.	Date of Commercial Operation	April 17, 1975 (92.6% of full power)
8.	Condenser Cooling Method	Closed Cycle
9.	Condenser Cooling Water Source	Natural Draft Cooling Towers - Folsom Canal

Utility & Contractor Information

10.	Licensee	Sacramento Municipal Utility District
11.	Corporate Address	6201 South Street P. O. Box 15830 Sacramento, California
12.	Corporate Contact	J. J. Mattimoe Assistant General Manager, Chief Engineer
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	B&W
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	V
17.	IE Branch Chief	J. L. Crews
18.	IE Senior Inspector	B. H. Faulkenberry
19.	IE Principal Inspector	H. L. Canter
20.	Licensing Project Manager	D. J. Garner
21.	Docket Number	50-312
22.	License Number/Date of Issue	DPR-54, 8/16/74
23.	Public Document Room	Business and Municipal Department Sacramento City-County Library 828 I Street Sacramento, California 95814

INSPECTION STATUS

ENFORCEMENT STATUS

TECHNICAL SPECIFICATION 6.5.2.7.I REQUIRES THAT THE MSRC REVIEW REPORTS AND MEETING MINUTES OF THE PLANT REVIEW COMMITTEE. CONTRARY TO THIS REQUIREMENT, IN THE PERIOD FROM JANUARY THROUGH AUGUST 1979,DURING WHICH THERE WERE APPROXIMATELY 75 MEETINGS OF THE PLANT REVIEW COMMITTEE, THE MINUTES AND AGENDA OF MSRC MEETINGS INDICATE THERE WERE NO INSTANCES WHEN THE MSRC REVIEWED THE MINUTES OF THE PLANT REVIEW COMMITTEE.
(7920 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST (CITE RO REPORT NUMBER, IF APP.)

SYSTEMS AND COMPONENT PROBLEMS:

None.

FACILITY ITEMS (PLANS AND PROCEDURES):

None.

MANAGERIAL ITEMS:

None.

PLANT STATUS:

Plant operating at 100% power.

LAST IE SITE INSPECTION DATE: 12/03-19/79+

INSPECTION REPORT NO. 50-312/79-28+

SUMMARY

INSPECTION SUMMARY

Inspection on July 24 - August 2, 1979 (Report No. 50-312/79-17)
Report being prepared; to be reported next month.

+ Inspection on October 29 - November 1, 1979 (Report No. 50-312/79-22)
Areas Inspected: Routine, unannounced inspection of personnel qualification program; design changes and modification program; test and experiments program; and independent inspection effort. The inspection involved 26 inspector hours onsite by one NRC inspector.

Results: Of the four areas inspected, no items of noncompliance or deviations were found in two areas. Three items of noncompliance were identified in two areas (Infraction - Failure to review temporary changes to procedures within seven days; Deficiency - Failure to audit qualifications of plant operating staff; Infraction - Failure to have temporary changes in procedures approved by two members of the plant management staff.

+ Inspection on November 21, 1979 (Report No. 50-312/79-24)
Report being prepared; to be reported next month.

+ Inspection on November 1-30, 1979 (Report No. 50-312/79-25)
Areas Inspected: Routine inspections of plant operations; physical security; training; requalification training; plant incidents; LER followup and reporting; review and audit; followup on inspector identified items; and independent inspection effort. The inspection involved 85 inspector hours onsite by the NRC Resident Inspector.

Results: No items of noncompliance were identified in seven areas. Two items of noncompliance were identified in two areas (plant incidents and LER followup and reporting).

+ Inspection on December 10-14, 1979 (Report No. 50-312/79-26)
Areas Inspected: Routine, unannounced inspection of the completion of selected fire protection modifications; surveillance of pipe support and restraint systems; followup on IE Bulletins and Circulars and audit of health physics and respirator training. The inspection involved 37 inspector hours onsite by one NRC inspector.

Results: No items of noncompliance or deviations were identified.

+ Inspection on December 10-14, 1979 (Report No. 50-312/79-27)
Areas Inspected: Security organization; physical barriers (protected area); physical barriers (vital areas); lighting; access control (identification, authorization and badging); access control (search); access control (escorting); access control (vital areas); alarm stations; detection aids; assessment aids; communications; testing and maintenance; compensatory measures; response; locks, keys and combinations; and records and reports. The inspection involved 212 inspector hours onsite by three NRC inspectors.

Results: No items of noncompliance or deviations were identified.

+ Inspection on December 3-19, 1979 (Report No. 50-312/79-28)
Report being prepared; to be reported next month.

NRC POWER RESTRICTIONS: None

REPORTS RECEIVED FROM LICENSEE

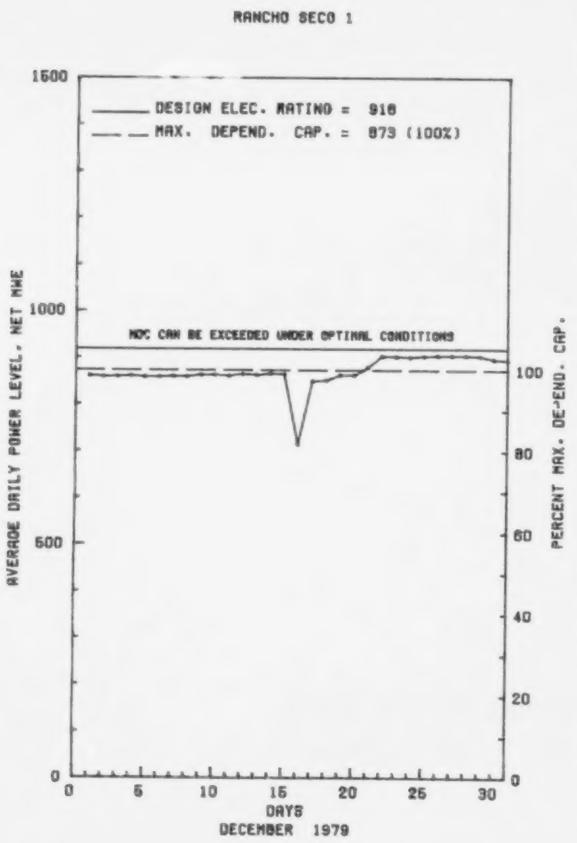
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-03-E1	11-29-79	12-12-79	Environmental	High chlorine concentration caused by debris clogging part of diverter.
79-04-E0	12-03-79	12-11-79	Environmental	Chlorine concentration of 1.6 MG/L exceeded Appendix B TS limit of 0.2 MG/L.
79-12-L1	10-05-79	11-05-79	30 Day LER	IE Bulletin 79-02 - Pipe support anchor bolt deficiencies.
79-14-L1	10-08-79	11-09-79	30 Day LER	Missing seismic support in NSCWS-Found in IB 79-14 inspection.
79-17-L0	11-09-79	12-04-79	30 Day LER	Diesel A failed to start on command from control room.
79-19-L0	11-25-79	12-10-79	30 Day LER	Pinhole leak in socket weld in seal injection line.
79-05-E0	12-05-79	12-13-79	Environmental	Chlorine concentration of 1 MG/L exceeded TS limit of 0.2 MG/L.
79-06-E0	12-14-79	12-24-79	Environmental	Chlorine concentration of 0.6 MG/L exceeded TS limit of 0.2 MG/L.

REVIEWED BY OMPA LOEB: PE Robe **DATE:** 2/11/80

REVIEWED BY NRR DOR: Daniel J Garner **DATE:** ______

RANCHO SECO

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-312
2. Utility Contact: R. W. Colombo (916) 452-3211
3. Licensed Thermal Power (MWt): 2772
4. Nameplate Rating (Gross MWe): 1070 x 0.9 = 963
5. Design Electrical Rating (Net MWe): 918
6. Maximum Dependable Capacity (Gross MWe): 917
7. Maximum Dependable Capacity (Net MWe): 873
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	8,760.0	41,257.0
12. Hours Reactor Was Critical	744.0	7,002.7	26,073.5
13. Reactor Reserve Shutdown Hours	.0	1,557.7	3,975.1
14. Hours Generator On-Line	744.0	6,778.1	24,941.3
15. Unit Reserve Shutdown Hours	.0	1,199.4	1,210.2
16. Gross Thermal Energy (MWH)	1,975,283	17,914,504	62,984,018
17. Gross Electrical Energy (MWH)	668,606	6,021,736	21,284,536
18. Net Electrical Energy (MWH)	636,415	5,711,999	20,110,890
19. Unit Service Factor	100.0	77.4	60.5
20. Unit Availability Factor	100.0	91.1	63.4
21. Unit Capacity Factor (MDC Net)	98.0	74.7	55.8
22. Unit Capacity Factor (DER Net)	93.2	71.0	53.1
23. Unit Forced Outage Rate	.0	5.6	32.1

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUELING: JANUARY 19, 1980 - APPROX. 60 DAYS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
20	12/16/79	S	.0	F	5				REDUCED POWER FOR AXIAL POWER SHAPING RODS WITHDRAWAL (STP-220).

SUMMARY

OPERATION WAS AT NEAR FULL POWER DURING THE REPORT PERIOD, WITH A LOAD REDUCTION ON 12/16/79 FOR AXIAL POWER SHAPING RODS WITHDRAWAL.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

ROBINSON 2

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	South Carolina/Darlington 5miles NW of Hartsville
2.	Electric Reliability Council	Southeastern
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2300
5.	Design Electrical Rating (MWE Net)	700
6.	Date of Initial Criticality	September 20, 1970
7.	Date of Commercial Operation	March 7, 1971
8.	Condenser Cooling Method	Recirculation from impounded lake
9.	Condenser Cooling Water Source	Robinson Impoundment

Utility & Contractor Information

10.	Licensee	Carolina Power and Light Company
11.	Corporate Address	411 Fayetteville Street Raleigh, North Carolina
12.	Corporate Contact	J. A. Jones Executive Vice President
13.	Architect/Engineer	Ebasco
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Ebasco

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R.C. Lewis
18.	IE Senior Inspector	P. J. Kellogg
19.	IE Principal Inspector	T. McHenry
20.	Licensing Project Manager	D. Neighbors
21.	Docket Number	50-261
22.	License Number/Date of Issue	DPR-23, 7/31/70 (5 Mwt), Full power authorization issued 9/23/70
23.	Public Document Room	Hartsville Memorial Library Home and Fifth Avenue Hartsville, South Carolina 29550

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO 10 CFR 50, APPENDIX B, CRITERION V, THE INSTRUCTIONS SPECIFIED IN IE BULLETIN #79-14 WERE NOT ADHERED TO DURING IE BULLETIN #79-14 INSPECTION. CP&L FAILED TO IDENTIFY THE DESIGN & LOCATION OF SEVERAL SUPPORTS & RESTRAINTS.
(7925 INFRACTION)

CONTRARY TO 10 CFR 50, APPENDIX B, CRITERION XVII, THE DATA AND INSPECTION RECORDS OBTAINED DURING THE ACCOMPLISHMENT OF IE BULLETIN #79-14 DO NOT IDENTIFY THE INSPECTOR OR DATA RECORDER.
(7925 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

PLANT OPERATING AT 100 PERCENT OF FULL POWER.

LAST IE SITE INSPECTION DATE: DECEMBER 11-14, 1979

INSPECTION REPORT NO: 50-261/79-30

SUMMARY

INSPECTION SUMMARY

INSPECTION NOVEMBER 13-14 (79-25): THIS SPECIAL, ANNOUNCED INSPECTION INVOLVED 26 INSPECTOR-HOURS ON SITE IN THE AREAS OF SEISMIC ANALYSIS FOR AS-BUILT SAFETY-RELATED PIPING SYSTEMS (IE BULLETIN 79-14). WITHIN THE AREA INSPECTED, TWO ITEMS OF NONCOMPLIANCE WERE FOUND (DEFICIENCY - INADEQUATE QA RECORDS TO IEB 79-14; INFRACTION - FAILURE TO IDENTIFY ALL PIPE SUPPORTS AND RESTRAINTS AS SPECIFIED IN IEB 79-14).

INSPECTION NOVEMBER 13-16 (79-26): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 22 INSPECTOR-HOURS ON SITE IN THE AREAS OF RADIOLOGICAL ENVIRONMENTAL MONITORING PROGRAM INCLUDING: MANAGEMENT CONTROLS, INSPECTION OF ENVIRONMENTAL MONITORING STATIONS, REVIEW OF LICENSEE AUDIT RESULTS, REVIEW OF ANNUAL ENVIRONMENTAL REPORT, REVIEW OF RADIOLOGICAL ENVIRONMENTAL MONITORING PROCEDURES, AND IMPLEMENTATION OF THE MONITORING PROGRAM. OF THE SIX AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

NRC POWER RESTRICTIONS: NONE;

REPORTS RECEIVED FROM LICENSEE

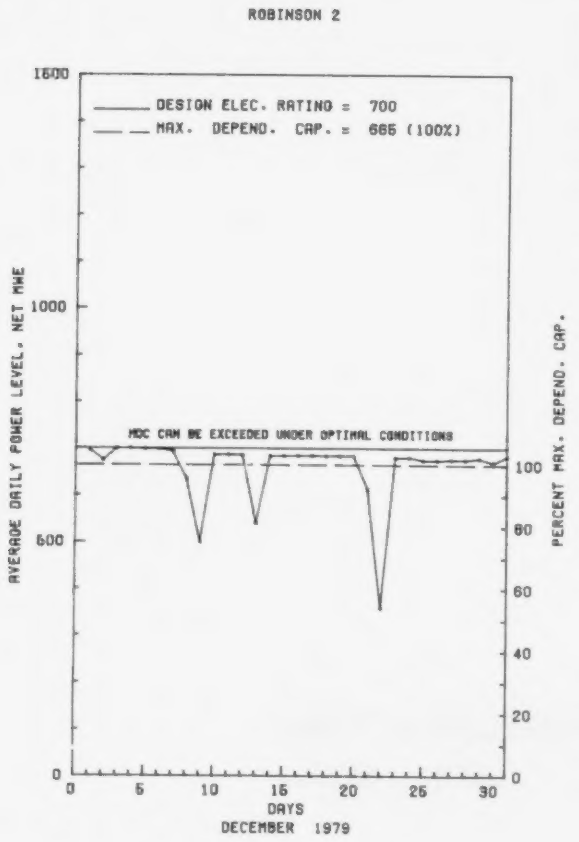
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-041/ 03L-0	10/30/79	11/29/79	30 DAY LER	WRONG CLASS OF INSULATING MATERIAL USED ON MOTOR LEADS OF ONE CONTAINMENT FAN COOLER.
79-042/ 03L-0	11/12/79	12/12/79	30 DAY LER	REPLACEMENT OF A SAFEGUARDS TEST SWITCH WITHOUT AN APPROVED OPERATING WORK PROCEDURE.

REVIEWED BY OMPA LOEB: T.C. Cintula DATE: 2.11.80
REVIEWED BY NRR DOR: Don Neighbors DATE: 2/13/80

GPO 871-667

ROBINSON 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-261
2. Utility Contact: M. L. Watford (803) 383-4524
3. Licensed Thermal Power (MWt): 2300
4. Nameplate Rating (Gross MWe): 854 x 0.9 = 769
5. Design Electrical Rating (Net MWe): 700
6. Maximum Dependable Capacity (Gross MWe): 700
7. Maximum Dependable Capacity (Net MWe): 665
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): 635
10. Reasons for Restrictions, If Any:
EXCESSIVE MOISTURE CARRY-OVER TO H.P. TURBINE.

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	8,760.0	77,382.0
12. Hours Reactor Was Critical	738.7	6,393.9	60,206.9
13. Reactor Reserve Shutdown Hours	5.3	39.2	719.3
14. Hours Generator On-Line	735.4	6,175.4	58,690.9
15. Unit Reserve Shutdown Hours	.0	23.2	23.2
16. Gross Thermal Energy (MWH)	1,555,205	12,969,260	120,550,329
17. Gross Electrical Energy (MWH)	515,593	4,222,412	38,936,513
18. Net Electrical Energy (MWH)	491,002	4,005,007	36,905,102
19. Unit Service Factor	98.8	70.5	75.8
20. Unit Availability Factor	98.8	70.8	75.9
21. Unit Capacity Factor (MDC Net)	99.2	68.8	71.7
22. Unit Capacity Factor (DER Net)	94.3	65.3	68.1
23. Unit Forced Outage Rate	1.2	4.2	13.2

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
REFUELING/MAINTENANCE, MAY 1980, 6 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
12-1	12/09/79	F	.0	B	5		HCC	HTEXCH	LOAD REDUCTION TO PLUG LEAKING CONDENSER TUBES.
12-2	12/13/79	F	2.9	G	3		HH	PUMPXX	REACTOR TRIP DUE TO "A" STEAM GENERATOR HI LEVEL. TRIP OCCURRED WHILE OPERATOR WAS VALVING IN 1530A, WHICH CAUSED THE LOSS OF BOTH HEATER DRAIN PUMPS.
12-3	12/21/79	F	.0	B	5		HC	HTEXCH	LOAD REDUCTION TO PLUG LEAKING CONDENSER TUBES.
12-4	12/22/79	F	5.7	G	3		HB	VALVEX	TURBINE TRIP DURING WEEKLY VALVE TEST. LEVER NOT IN FULL TEST POSITION.

SUMMARY

OPERATION WAS AT NEAR FULL POWER DURING THE REPORT PERIOD, WITH LOAD REDUCTIONS ON 12/09/79 AND 12/21/79 TO PLUG LEAKING CONDENSER TUBES. A REACTOR TRIP DUE TO "A" STEAM GENERATOR HIGH LEVEL OCCURRED ON 12/13/79, AND A TURBINE TRIP DURING A VALVE TEST OCCURRED ON 12/22/79 FROM THE LEVER NOT BEING IN THE FULL TEST POSITION.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

SALEM 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1. Location (state/countv) (nearest population center)	New Jersey/Salem 20 miles S of Wilmington, Del.
2. Electric Reliability Council	NPCC
3. Type of Reactor	PWR
4. Power Level (MWT)	3338
5. Design Electrical Rating (MWE Net)	1090
6. Date of Initial Criticality	December 11, 1976
7. Date of Commercial Operation	June 30, 1977
8. Condenser Cooling Method	Once Through
9. Condenser Cooling Water Source	Delaware River

Utility & Contractor Information

10. Licensee	Public Service Electric & Gas Co.
11. Corporate Address	80 Park Place Newark, New Jersey 07101
12. Corporate Contact	F. P. Librizzi General Manager - Electric Prod.
13. Architect/Engineer	PSE&G
14. Nuclear Steam Supply System	Westinghouse
15. Constructor	UE&C

Regulatory Information

16. IE Region responsible	I
17. IE Branch Chief	E. Brunner
18. IE Senior Inspector	R. R. Keimig
19. IE Resident Inspector	L. J. Norrholm (609-935-3850) (FTS 341-3000)
20. Licensing Project Manager	W. Ross
21. Docket Number	50-272
22. License Number/Date of Issue	DPR-70, 8/13/76
23. Public Document Room	Salem Free Public Library 112 West Broadway Salem, New Jersey 08079

INSPECTION STATUS

ENFORCEMENT STATUS

THE 16.5 DEGREE F MAXIMUM T ACROSS THE CONDENSER WAS EXCEEDED DURING NORMAL OPERATION WITH ALL CIRCULATING WATER PUMPS OPERATING ON SEVERAL OCCASIONS INCLUDING 1/4, 5, 23 & 25; 2/5-6 & 3/4/79. THE 27.5 DEGREE F INSTANTANEOUS MAXIMUM T ACROSS THE CONDENSER WAS EXCEEDED ON 1/18 & 25/79. SECTION 2.1.1 OF APP B, ETS FOR SALEM GENERATING STATION UNIT 1.
(7929 INFRACTION)

RECORDS OF PLANT THERMAL MONITORING REQUIRED BY SECTION 2.1 OF THE ETS WERE NOT MADE & RETAINED FOR THE PERIOD FROM 0900 THRU 2400 HRS ON 1/13/79. SECTION 5.7.1 OF APP B, ETS.
(7929 DEFICIENCY)

ON SEVERAL OCCASIONS, INCLUDING THOSE INSTANCES INDICATED IN ITEM A OF THIS APP, THE EXCEEDING OF THE CONDENSER T LIMITING CONDITION FOR OPERATION INCLUDING THOSE OF 1/4, 5, 23 & 25; 2/5-6 & 3/4/79 WERE NOT REPORT AS REQUIRED. SECTION 5.6.2 OF APP B, ETS.
(7929 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

- None

FACILITY ITEMS (PLANS AND PROCEDURES):

- The licensee is completing seismic reanalysis per IEB 79-07 and making support modifications as necessary.

MANAGERIAL ITEMS:

None

PLANT STATUS:

- Unit resumed operation at the end of December.
- LAST IE SITE INSPECTION DATE: 01/22/80
- INSPECTION REPORT NO: 79-33

SUMMARY

INSPECTION SUMMARY

- Oct 7 - Nov 17 (79-28): Routine inspections of plant operations including: tours of the facility; log and record reviews, review of licensee events; followup on IE Bulletins and Immediate Action Letters; and followup on previous inspection items. The inspections involved 75 onsite hours by the resident NRC inspector. One item of noncompliance and one deviation were identified (failure to follow procedures relative to valve status; failure to complete seismic support modifications prior to Mode 4 operation).
- Oct 31 - Nov 1-2 (79-30): Routine unannounced inspection of inservice inspection data package and licensee action on previous inspection findings. The inspection involved 24 hours onsite by one NRC region-based inspector. No noncompliances were identified.

REPORTS RECEIVED FROM LICENSEE

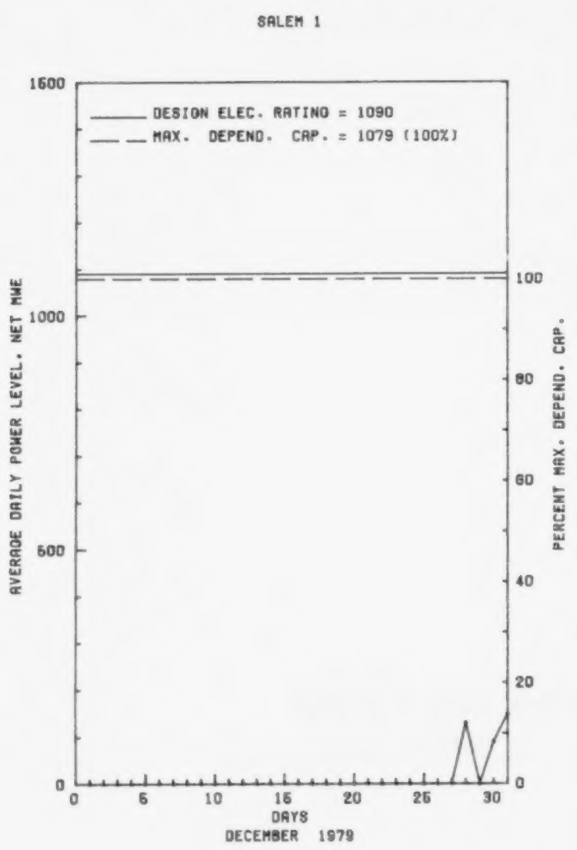
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
	NONE			

REVIEWED BY OMPA LOEB: T. C. Cintula DATE: 2-11-80

REVIEWED BY NRR DOR: W J Ross DATE: 2/13/80

SALEM 1

AVERAGE DAILY POWER LEVEL(MWe)



OPERATING STATUS

1. Docket Number: 50-272
2. Utility Contact: L. K. Miller (609) 365-7000
3. Licensed Thermal Power (MWt): 3338
4. Nameplate Rating (Gross MWe): 1300 x 0.9 = 1170
5. Design Electrical Rating (Net MWe): 1090
6. Maximum Dependable Capacity (Gross MWe): 1124
7. Maximum Dependable Capacity (Net MWe): 1079
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	8,760.0	21,961.0
12. Hours Reactor Was Critical	246.9	2,431.4	10,088.3
13. Reactor Reserve Shutdown Hours	.0	22.7	22.7
14. Hours Generator On-Line	71.8	2,233.1	9,531.1
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	69,070	6,551,561	27,536,296
17. Gross Electrical Energy (MWH)	12,920	2,196,690	9,167,500
18. Net Electrical Energy (MWH)	-2,261	2,062,610	8,629,590
19. Unit Service Factor	9.7	25.5	43.4
20. Unit Availability Factor	9.7	25.5	43.4
21. Unit Capacity Factor (MDC Net)	.0	21.6	36.6
22. Unit Capacity Factor (DER Net)	.0	21.4	36.1
23. Unit Forced Outage Rate	90.3	61.0	44.8

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
79-329	12/01/79	F	.0	D	5		SH	ZZZZZZ	SAFETY ENVIRONMENTAL REQUIREMENTS.
79-350	12/01/79	F	101.5	D	9		SH	ZZZZZZ	NRC TECH. SPECS./REG. GUIDES.
79-352	12/05/79	F	537.8	D	9		SH	ZZZZZZ	NRC REQUIRED MOD/ADD. TO PLANT.
79-354	12/27/79	F	.0	D	5		SH	ZZZZZZ	NRC TECH. SPECS./REG. GUIDES.
79-356	12/27/79	F	.5	D	9		SH	ZZZZZZ	SWITCHGEAR.
79-358	12/27/79	F	.0	D	5		SH	ZZZZZZ	NRC TECH. SPECS./REG. GUIDES.
79-360	12/29/79	F	18.8	D	9		SH	ZZZZZZ	REQUIRED INSPECTION OR TEST.
79-362	12/30/79	F	.0	D	5		SH	ZZZZZZ	NRC TECH. SPECS./REG. GUIDES.
79-364	12/30/79	F	6.0	D	9		SH	ZZZZZZ	REQUIRED INSPECTION OR TEST.
79-365	12/30/79	F	.0	D	5		SH	ZZZZZZ	NRC TECH. SPECS./REG. GUIDES.
79-366	12/31/79	F	7.6	D	3		CC	PUMPXX	STEAM GENERATOR HIGH HIGH LEVEL.

SUMMARY

THE UNIT REMAINED SHUTDOWN FOR MOST OF THE REPORT PERIOD AS A COMBINATION OF LAST PERIOD'S MAINTENANCE OUTAGE. LOAD INCREASES OCCURRED ON 12/28/79 FOR FLUX MAPPING, AND ON 12/31/79 IN PREPARATION FOR POWER OPERATION.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

SAN ONOFRE 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	California/San Diego 5 miles S of San Clemente
2.	Electric Reliability Council	WSCC
3.	Type of Reactor	PWR
4.	Power Level (MWT)	1347
5.	Design Electrical Rating (MWE Net)	436
6.	Date of Initial Criticality	June 14, 1967
7.	Date of Commercial Operation	January 1, 1968
8.	Condenser Cooling Method	Circulating sea water
9.	Condenser Cooling Water Source	Pacific Ocean

Utility & Contractor Information

10.	Licensee	Southern California Edison Co.
11.	Corporate Address	2244 Walnut Grove Avenue Rosemead, California 91770
12.	Corporate Contact	R. E. Woodbury V.P. & General Counsel
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	V
17.	IE Branch Chief	J. L. Crews
18.	IE Senior Inspector	B. H. Faulkenberry
19.	IE Principal Inspector	L. G. Miller
20.	Licensing Project Manager	A. Burger
21.	Docket Number	50-206
22.	License Number/Date of Issue	DPR-13, 3/27/67
23.	Public Document Room	Mission Viejo Branch Library 24851 Chrisanta Drive Mission Viejo, California 92676

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST (CITE RO REPORT NUMBER, IF APP.)

SYSTEMS AND COMPONENT PROBLEMS:

None.

FACILITY ITEMS (PLANS AND PROCEDURES):

None.

MANAGERIAL ITEMS

None.

PLANT STATUS:

Plant operating at 100% power.

LAST IE SITE INSPECTION DATE: 12/10-13/79+

INSPECTION REPORT NO: 50-206/79-17+

SUMMARY

INSPECTION SUMMARY

+ Inspection on October 15-19 & 30 & November 2, 1979 (Report No. 50-206/79-15)
Areas Inspected: Routine, unannounced inspection of the radioactive waste handling program, including review of release records, methods and procedures for treatment of solid, liquid and gaseous radioactive waste; examination of effluent monitoring systems and sampling methods; and collection of confirmatory measurement samples. The inspector investigated the licensee action to date in response to IE Bulletin 79-19 and two Licensee Event Reports (LER 79-11 and 12) related to environmental monitoring. The status of emergency response planning was discussed and an emergency drill was observed. The inspection involved a total of 53 hours onsite by one NRC inspector.

Results: No items of noncompliance or deviations were identified.

+ Inspection on November 1-2 & 26, 1979 (Report No. 50-206/79-16)
Areas Inspected: Routine, announced inspection of scope and methods of field inspection for IE Bulletin 79-14; routine, unannounced inspection of repair of shorted electrical buswork from the November 7, 1979 reportable occurrence; discussions of this event, of the proposed licensee design change 79-20, and of the recent reportable occurrences (LER 79-18 and 79-19) involving missing pipe supports. The inspection involved 47 inspector hours by three NRC inspectors.

Results: No items of noncompliance or deviations were identified.

+ Inspection on December 10-13, 1979 (Report No. 50-206/79-17)
Areas Inspected: Routine, unannounced inspection of quality assurance program; onsite review; procedures; plant operations; plant tour;

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-20-LO	11-26-79	12-07-79	30 Day LER	Missing and misinstalled valve operator supports on seal water return.
79-21-LO	11-16-79	12-12-79	30 Day LER	OSRC discovered post-facto that one train of ECCS disabled for seven hours.
79-22-LO	11-30-79	12-17-79	30 Day LER	Missing supports on CCW supply to cont. sphere.
79-23-LO	10-10-79	12-11-79	30 Day LER	QA audit discovered PRZR PSR trip set slightly too low.

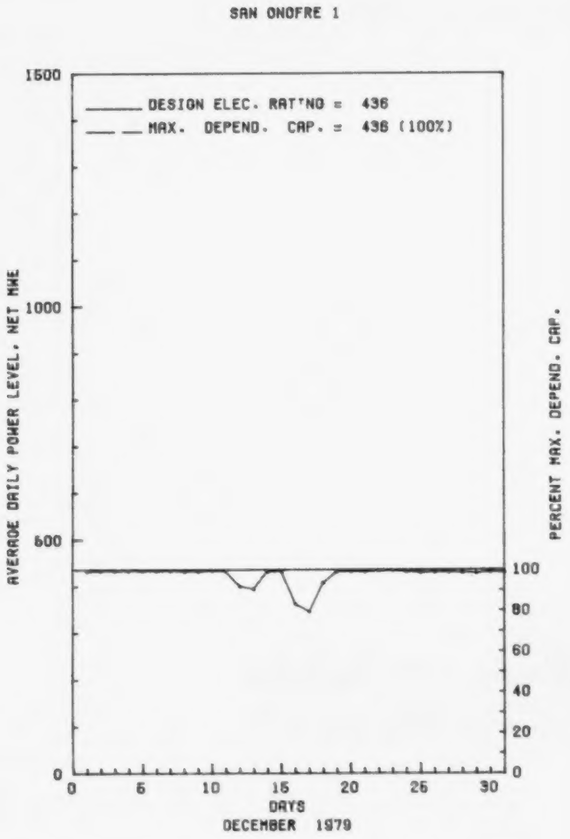
REVIEWED BY OMPA LOEB: T.C. CINTULA DATE: 2011080

REVIEWED BY NRR DOR: ______________ DATE: ________

GPO 878-667

SAN ONOFRE 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-206
2. Utility Contact: R. G. Moran (714) 492-7700
3. Licensed Thermal Power (MWt): 1347
4. Nameplate Rating (Gross MWe): 500 x 0.9 = 450
5. Design Electrical Rating (Net MWe): 436
6. Maximum Dependable Capacity (Gross MWe): 456
7. Maximum Dependable Capacity (Net MWe): 436
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	8,760.0	109,976.0
12. Hours Reactor Was Critical	744.0	7,916.3	82,632.2
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	744.0	7,905.0	79,152.4
15. Unit Reserve Shutdown Hours	.0	0	.0
16. Gross Thermal Energy (MWH)	986,706	10,522,812	101,531,474
17. Gross Electrical Energy (MWH)	330,600	3,520,200	34,671,434
18. Net Electrical Energy (MWH)	317,135	3,355,531	32,847,366
19. Unit Service Factor	100.0	90.2	75.5
20. Unit Availability Factor	100.0	90.2	75.5
21. Unit Capacity Factor (MDC Net)	97.8	87.9	73.0
22. Unit Capacity Factor (DER Net)	97.8	87.9	73.0
23. Unit Forced Outage Rate	.0	5.4	9.3

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): TMI BACKFIT - 01/21/80; REFUELING - 04/80.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
37	12/12/79	S	8.3	B	5		HF	HTEXCH	CONDENSER TUBE LEAK
38	12/16/80	F	50.7	A	5		HF	PUMPXX	CIRCULATING WATER PUMP FAILURE.

SUMMARY

OPERATION WAS AT NEAR FULL POWER DURING THE REPORT PERIOD, WITH LOAD REDUCTIONS ON 12/12/79 TO LOCATE A CONDENSER TUBE LEAK, AND ON 12/16/79 TO REPAIR THE SOUTH CIRCULATING WATER PUMP.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

ST. LUCIE 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Florida/St. Lucie 12 miles SE of Ft. Pierce
2.	Electric Reliability Council	Southern
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2560
5.	Design Electrical Rating (MWE Net)	802
6.	Date of Initial Criticality	April 22, 1976
7.	Date of Commercial Operation	December 21, 1976
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Atlantic Ocean

Utility & Contractor Information

10.	Licensee	Florida Power & Light Co.
11.	Corporate Address	9250 W. Flagler Street P. O. Box 529100 Miami, Florida 33152
12.	Corporate Contact	R. E. Uhrig, V.P.
13.	Architect/Engineer	Ebasco
14.	Nuclear Steam Supply System	Combustion Engineering
15.	Constructor	Ebasco

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. Lewis
18.	IE Senior Inspector	E.H. Verdery
19.	IE Principal Inspector	J.A. Dyer
20.	Licensing Project Manager	P. B. Erickson
21.	Docket Number	50-335
22.	License Number/Date of Issue	DPR-67, March 1, 1976
23.	Public Document Room	Indian River Junior College Library 3209 Virginia Avenue Ft. Pierce, Florida 33450

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

```
INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

UNIT OPERATING AT 100% POWER.

*    LAST IE SITE INSPECTION DATE:  DECEMBER 10-13, 1979

*    INSPECTION REPORT NO:  50-335/79-32

     SUMMARY

     INSPECTION SUMMARY

*    INSPECTION NOVEMBER 26-28 (79-30):  THIS ROUTINE, UNANNOUNCED INSPECTION
     INVOLVED 21 INSPECTOR-HOURS ON SITE IN THE AREAS OF LICENSEE ACTION ON
     PREVIOUSLY IDENTIFIED QUALITY ASSURANCE ITEMS AND QA PROCUREMENT.  OF
     THE TWO AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE
     IDENTIFIED.

     INSPECTION NOVEMBER 14-16 (79-31):  THIS SPECIAL, UNANNOUNCED INSPECTION
     INVOLVED 13 INSPECTOR-HOURS ON SITE IN THE AREA OF PIPE SUPPORT BASE
     PLATE DESIGNS USING CONCRETE EXPANSION ANCHOR BOLTS (IE BULLETIN 79-02).
     NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

     INSPECTION NOVEMBER 27-30 (79-34):  THIS ROUTINE UNANNOUNCED INSPECTION
     INVOLVED 24 INSPECTOR-HOURS ON SITE IN THE AREAS OF REACTOR COOLANT LOOP
     PIPING WELDING ACTIVITIES, WELDING MATERIAL CONTROL AND WELDER QUALIFICA-
     TION (UNIT 2); FOLLOWUP INSPECTION OF AN UNRESOLVED ITEM, ITEM OF NONCOM-
     PLIANCE AND AN INSPECTOR FOLLOWUP ITEM (UNIT 2); FOLLOWUP OF IE BULLETIN
     79-13 (UNIT 1).  OF THE FOUR AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE
     OR DEVIATIONS WERE IDENTIFIED.
```

REPORTS RECEIVED FROM LICENSEE

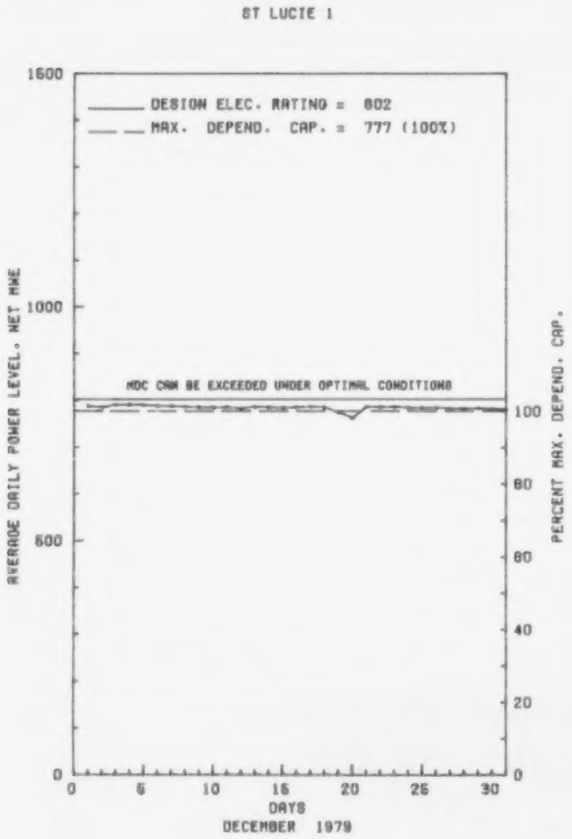
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-04/ 03L-0	11/13/79	12/12/79	30 DAY LER	OCEAN INTAKE RECORDING THERMOGRAPH MOUNTING ARRAY AND FLOAT ATGON BOUY MISSING.

REVIEWED BY OMPA LOEB: AR Beebe **DATE:** 02/11/80

REVIEWED BY NRR DOR: P B Erickson **DATE:** 2/12/80

ST. LUCIE 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-335
2. Utility Contact: V. T. Chilson (305) 552-3824
3. Licensed Thermal Power (MWt): 2560
4. Nameplate Rating (Gross MWe): 1000 x 0.85 = 850
5. Design Electrical Rating (Net MWe): 802
6. Maximum Dependable Capacity (Gross MWe): 822
7. Maximum Dependable Capacity (Net MWe): 777
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	8,760.0	26,544.0
12. Hours Reactor Was Critical	744.0	6,722.3	21,509.9
13. Reactor Reserve Shutdown Hours	.0	11.0	129.4
14. Hours Generator On-Line	744.0	6,470.9	20,838.1
15. Unit Reserve Shutdown Hours	.0	11.0	32.0
16. Gross Thermal Energy (MWH)	1,901,210	16,041,418	50,259,566
17. Gross Electrical Energy (MWH)	616,300	5,187,550	16,294,500
18. Net Electrical Energy (MWH)	583,458	4,885,058	15,327,896
19. Unit Service Factor	100.0	73.9	78.5
20. Unit Availability Factor	100.0	74.0	78.6
21. Unit Capacity Factor (MDC Net)	100.9	71.8	74.3
22. Unit Capacity Factor (DER Net)	97.8	69.5	72.0
23. Unit Forced Outage Rate	.0	3.2	5.4

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
REFUEL, MAINT. & INSPEC. - 03/16-05/10/80.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	NONE								

SUMMARY

OPERATION WAS AT NEAR FULL POWER THROUGHOUT THE REPORT PERIOD, WITH BRIEF REDUCTIONS FOR MAINTENANCE.

LEGEND

[1] Type:
F–Forced
S–Scheduled

[2] Reason:
A–Equipment Failure
B–Maintenance or Test
C–Refueling
D–Regulatory Restriction
E–Operator Training & License Examination
F–Administrative
G–Operational Error
H–Other

[3] Method:
1–Manual
2–Manual Scram
3–Automatic Scram
4–Continuations
5–Load Reduction
9–Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

SURRY 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Virginia/Surry 17 miles NW of Newport News
2.	Electric Reliability Council	Southeastern
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2441
5.	Design Electrical Rating (MWE Net)	822
6.	Date of Initial Criticality	July 1, 1972
7.	Date of Commercial Operation	December 22, 1972
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	James River

Utility & Contractor Information

10.	Licensee	Virginia Electric and Power Company
11.	Corporate Address	P. O. Box 26666 Richmond, Virginia 23261
12.	Corporate Contact	W. L. Proffitt Senior Vice President
13.	Architect/Engineer	Stone & Webster
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R.C. Lewis
18.	IE Senior Inspector	P. J. Kellogg
19.	IE Resident Inspector	D. Burke
20.	Licensing Project Manager	D. Neighbors
21.	Docket Number	50-280
22.	License Number/Date of Issue	DPR-32, 5/25/72
23.	Public Document Room	Swem Library College of William and Mary Williamsburg, Virginia 23185

INSPECTION STATUS

ENFORCEMENT STATUS

AS REQUIRED BY TECHNICAL SPECIFICATION 3.1.B, THE REACTOR COOLANT SYSTEM (RCS) COOLDOWN RATE SHALL NOT EXCEED 50 DEGREES F PER HOUR FOR TEMPERATURES BELOW 440 DEGREES F. CONTRARY TO THE ABOVE, ON OCTOBER 5, 1978, THE RCS COOLDOWN RATE, AS MEASURED BY THE WIDE RANGE HOT LEG TEMPERATURE RECORDER TR-1-413, EXCEEDED THE LIMIT BY SOME 10 DEGREES F FOR THE FOLLOWING ONE HOUR PERIODS 3:30 P.M. TO 4:30 P.M., 390 DEGREES F TO 330 DEGREES F AND 7 P.M. TO 8 P.M., 260 DEGREES F TO 200 DEGREES F.
(7962 INFRACTION)

AS REQUIRED BY TECHNICAL SPECIFICATION 3.7, TABLE 3.7-5, ITEM 2, THE COMPONENT COOLING WATER RADIATION MONITORS RM-CC-105 AND -106 SHALL BE SET TO ALARM AT TWICE BACKGROUND OR LESS. CONTRARY TO THE ABOVE, ON OCTOBER 29, 1979, RM-CC-105 AND RM-CC-106 RADIATION MONITORS WERE SET TO ALARM AT APPROXIMATELY TEN TIMES BACKGROUND DUE TO THE DECREASE IN ACTIVITY IN THE COMPONENT COOLING WATER; THE ALARM SETPOINT WAS NOT REDUCED AS THE BACKGROUND DECREASED.
(7962 INFRACTION)

AS REQUIRED BY TS 6.4.0, THE DETAILED WRITTEN PROCEDURES FOR ACTIONS TO BE TAKEN FOR SPECIFIC AND UNFORESEEN MALFUNCTIONS OF SYSTEMS OR COMPONENTS, INCLUDING ALARMS, SHALL BE FOLLOWED. CONTRARY TO THE ABOVE, ON OCTOBER 17, 1979, ANNUNCIATOR ALARM PROCEDURE 18-26 (D-2) "RECIRCULATION SPRAY PIT B HIGH LEVEL" WAS NOT FOLLOWED AFTER THE INSPECTOR VERIFIED EXCESSIVE WATER (OVER 14 INCHES) IN THE SAFEGUARDS VALVE SUMP END PIT AND THE ALARMED ANNUNCIATOR D-2 ON THE UNIT 1 CONTROL ROOM PANEL B. ALTHOUGH THE ALARM HAD BEEN ANNUNCIATED FOR SEVERAL DAYS AND NOT DOCUMENTED, OPERATOR ACTION TO VERIFY THE WATER LEVELS, THE SOURCE OF THE WATER, THE PROPER OPERATION OF THE DRAIN AND SUMP PUMPS, AND TO REMOVE THE WATER WAS NOT TAKEN.
(7962 INFRACTION)

TABLE E-3 IN PART IV OF THE SURRY FIRE PROTECTION SYSTEMS REVIEW AND REVISIONS STATE THAT THE SPECIFIED PORTABLE FIRE EXTINGUISHERS ARE INSTALLED AND MAINTAINED IN ACCORDANCE WITH NFPA 10, WHICH REQUIRES MONTHLY INSPECTIONS (SECTION 4-3.1). CONTRARY TO THE ABOVE, ON OCT 25, 1979, ONE OF THE TWO PORTABLE FIRE EXTINGUISHERS IN EACH OF THE SAFEGUARDS VALVE PIT AREAS FOR UNITS 1 AND 2 WERE FOUND NOT INSPECTED SINCE JANUARY, 1979. (FIRE EXTINGUISHERS #94 AND #131).
(7962 DEVIATION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

RE-ANALYSIS AND REVIEW OF SEISMIC DESIGN DEFICIENCIES ARE CONTINUING.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

UNIT SHUTDOWN FOR REPAIRS.

* LAST IE SITE INSPECTION DATE: DECEMBER 14, 1979

* INSPECTION REPORT NO: 50-280/79-67

SUMMARY

INSPECTION SUMMARY

* INSPECTION OCTOBER 1-31 (79-62): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 90 INSPECTOR-HOURS ON SITE IN THE AREAS OF PLANT OPERATIONS AND OPERATIONS AND OPERATING RECORDS, PLANT MAINTENANCE, AND PLANT SECURITY. OF THE THREE AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN ONE AREA; TWO APPARENT ITEMS OF NONCOMPLIANCE WERE FOUND IN ONE AREA (INFRACTION - FAILURE TO FOLLOW ALARM PROCEDURE; INFRACTION - COOLDOWN RATE EXCEEDED TS LIMIT; ONE NONCOMPLIANCE AND ONE DEVIATION WERE FOUND IN THE REMAINING AREA; INFRACTION ALARM SET POINTS ON RADIATION MONITORS CC-105 AND 106 NOT IN ACCORDANCE WITH TS 3.7; DEVIATION - TWO PORTABLE FIRE EXTINGUISHERS NOT INSPECTED MONTHLY).

REPORTS RECEIVED FROM LICENSEE

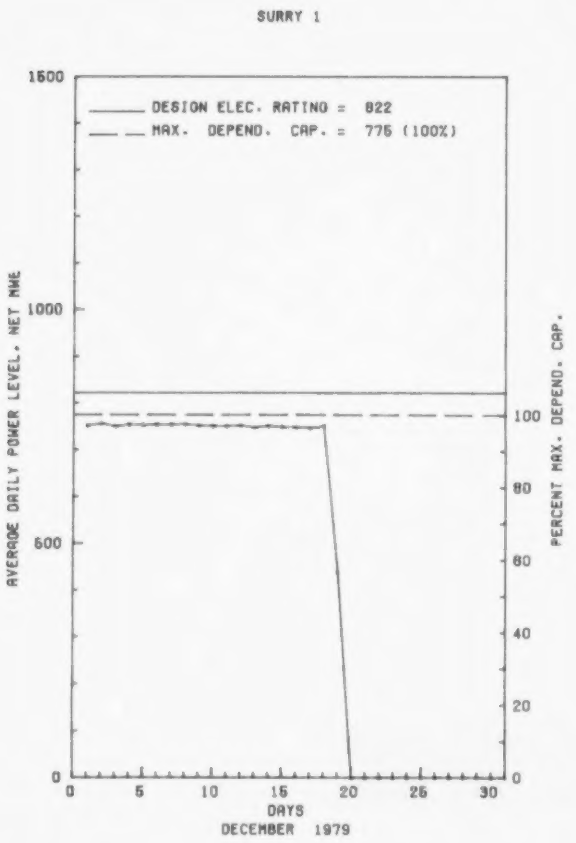
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-030/01T-0	10/29/79	11/09/79	14 DAY LER	RADIATION ALARM SETPOINT FOR COMPONENT COOLING WATER SET TOO HIGH.
79-031/03L-0	11/17/79	12/11/79	30 DAY LER	LIQUID WASTE RADIATION MONITOR FAILED TO ALARM AND TRIP SHUT THE RELEASE VALVE WHEN REQUIRED.
79-036/01T-0	11/28/79	12/11/79	14 DAY LER	ERROR FOUND IN ANALYSIS FOR CONTAINMENT SPRAY EFFECTIVENESS MODEL.
79-037/03L-0	11/19/79	12/11/79	30 DAY LER	FAILURE OF HEAT TRACE CIRCUIT 13A - BORIC ACID TANK RECIRCULATION LINES.

REVIEWED BY OMPA LOEB: DATE: 2/11/80

REVIEWED BY NRR DOR: Don Neighbors DATE: 2/13/80

SURRY 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-280
2. Utility Contact: O. J. Costello (804) 357-3184
3. Licensed Thermal Power (MWt): 2441
4. Nameplate Rating (Gross MWe): 942 x 0.9 = 848
5. Design Electrical Rating (Net MWe): 822
6. Maximum Dependable Capacity (Gross MWe): 811
7. Maximum Dependable Capacity (Net MWe): 775
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	8,760.0	61,584.0
12. Hours Reactor Was Critical	446.0	3,051.7	38,751.6
13. Reactor Reserve Shutdown Hours	68.0	3,548.9	3,548.9
14. Hours Generator On-Line	445.7	3,046.1	37,905.0
15. Unit Reserve Shutdown Hours	48.0	3,552.6	3,552.6
16. Gross Thermal Energy (MWH)	1,088,483	7,358,919	87,714,872
17. Gross Electrical Energy (MWH)	352,695	2,372,915	28,680,498
18. Net Electrical Energy (MWH)	335,501	2,255,180	27,226,899
19. Unit Service Factor	59.9	34.8	61.6
20. Unit Availability Factor	66.4	75.3	67.3
21. Unit Capacity Factor (MDC Net)	58.2	33.2	57.0
22. Unit Capacity Factor (DER Net)	54.9	31.3	53.8
23. Unit Forced Outage Rate	40.1	65.2	24.8

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): 06/01/80 - 6 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: NO REPORT

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
79-10	12/19/79	F	250.0	A	3	79-041			REACTOR TRIP CAUSED BY FAILURE OF IA REACTOR COOLANT PUMP MOTOR ON GROUND FAULT. MOTOR WILL BE REPLACED PRIOR TO STARTUP.
79-11	12/30/79	F	48.0	D	4				UNIT DOWN FOR REPLACEMENT OF RCP MOTOR. NRC REQUIREMENT TO SUCCESSFULLY TEST RCP SNUBBER PRIOR TO STARTUP.

SUMMARY

OPERATION WAS AT NEAR FULL POWER UNTIL 12/19/79 WHEN THE IA REACTOR COOLANT PUMP MOTOR FAILED ON GROUND FAULT. THE UNIT REMAINED SHUTDOWN FOR THE REMAINDER OF THE PERIOD, PENDING REPLACEMENT OF THE MOTOR.

LEGEND

[1] Type:
F-Forced
S-Scheduled

[2] Reason:
A-Equipment Failure
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error
H-Other

[3] Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Continuations
5-Load Reduction
9-Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

SURRY 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Virginia/Surry 17 miles NW of Newport News
2.	Electric Reliability Council	Southeastern
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2441
5.	Design Electrical Rating (MWE Net)	822
6.	Date of Initial Criticality	March 7, 1973
7.	Date of Commercial Operation	May 1, 1973
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	James River

Utility & Contractor Information

10.	Licensee	Virginia Electric and Power Company
11.	Corporate Address	P. O. Box 26666 Richmond, Virginia 23261
12.	Corporate Contact	W. L. Proffitt Senior Vice President
13.	Architect/Engineer	Stone & Webster
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R.C. Lewis
18.	IE Senior Inspector	P. J. Kellog
19.	IE Resident Inspector	D. Burke
20.	Licensing Project Manager	D. Neighbors
21.	Docket Number	50-281
22.	License Number/Date of Issue	DPR-37, 1/29/73
23.	Public Document Room	Swem Library College of William and Mary Williamsburg, Virginia 23185

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO CRITERION II OF APPENDIX B TO 10 CFR 50, CLEANLINESS MEASURES WERE NOT MAINTAINED IN ACTIVITIES AFFECTING QUALITY.
(7981 INFRACTION)

CONTRARY TO CRITERION XIII OF APPENDIX B TO 10 CFR 50, HANDLING TECHNIQUES USED IN ACTIVITIES AFFECTING QUALITY WERE NOT PROPERLY CONTROLLED.
(7981 INFRACTION)

TABLE E-3 IN PART IV OF THE SURRY FIRE PROTECTION SYSTEMS REVIEW AND REVISIONS STATE THAT THE SPECIFIED PORTABLE FIRE EXTINGUISHERS ARE INSTALLED AND MAINTAINED IN ACCORDANCE WITH NFPA 10, WHICH REQUIRES MONTHLY INSPECTIONS (SECTION 4-3.1). CONTRARY TO THE ABOVE, ON OCT. 25, 1979, ONE OF THE TWO PORTABLE FIRE EXTINGUISHERS IN EACH OF THE SAFEGUARDS VALVE PIT AREAS FOR UNITS 1 AND 2 WERE FOUND NOT INSPECTED SINCE JANUARY, 1979. (FIRE EXTINGUISHERS #94 AND #131).
(7982 DEVIATION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

STEAM GENERATOR REPAIR PROGRAM IS CONTINUING.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

- UNIT IS CURRENTLY SHUT DOWN AND DEFUELED. THE STEAM GENERATOR REPLACEMENT PROJECT COMPLETED.
- LAST IE SITE INSPECTION DATE: DECEMBER 14, 1979
- INSPECTION REPORT NO: 50-281/79-89

SUMMARY

INSPECTION SUMMARY

- INSPECTION OCTOBER 31 - NOVEMBER 2 (79-81): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 18 INSPECTOR-HOURS ON SITE IN THE AREAS OF CONSTRUCTION STATUS, ENGINEERING TASK ASSIGNMENT REVIEW AND OBSERVATION OF SAFETY-RELATED WORK ACTIVITIES. OF THE THREE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN TWO AREAS; TWO ITEMS OF NONCOMPLIANCE WERE FOUND IN ONE AREA (INFRACTION - IMPROPER RIGGING TECHNIQUES; AND INFRACTION - REACTOR COOLANT PUMP OPEN FLANGED NIPPLES).
- INSPECTION OCTOBER 1-31 (79-82): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 90 INSPECTOR-HOURS ON SITE IN THE AREAS OF PLANT OPERATIONS AND OPERATIONS AND OPERATING RECORDS, PLANT MAINTENANCE, AND PLANT SECURITY. OF THE THREE AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN ONE AREA; TWO APPARENT ITEMS OF NONCOMPLIANCE WERE FOUND IN ONE AREA (INFRACTION - FAILURE TO FOLLOW ALARM PROCEDURES; INFRACTION ALARM SET POINTS ON RADIATION MONITORS CC-105 AND 106 NOT IN ACCORDANCE WITH TS 3.7; DEVIATION - TWO PORTABLE FIRE EXTINGUISHERS NOT INSPECTED MONTHLY).

REPORTS RECEIVED FROM LICENSEE

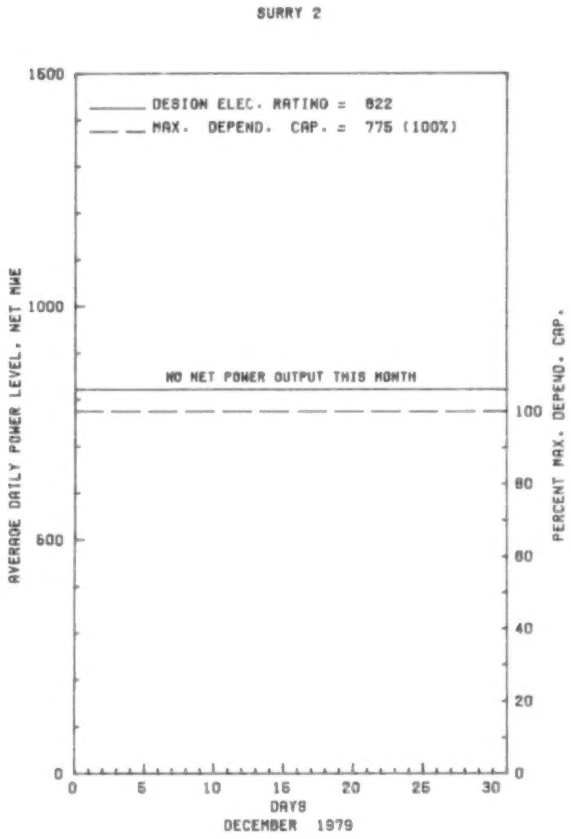
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
None				

REVIEWED BY OMPA LOEB: DATE: [illegible]

REVIEWED BY NRR DOR: Don Neighbors DATE: 2/13/80

SURRY 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-281
2. Utility Contact: O. J. Costello (804) 357-3184
3. Licensed Thermal Power (MWt): 2441
4. Nameplate Rating (Gross MWe): 942 x 0.9 = 848
5. Design Electrical Rating (Net MWe): 822
6. Maximum Dependable Capacity (Gross MWe): 811
7. Maximum Dependable Capacity (Net MWe): 775
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	8,760.0	58,464.0
12. Hours Reactor Was Critical	.0	819.6	36,499.9
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	.0	818.9	33,996.2
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	1,957,986	79,196,083
17. Gross Electrical Energy (MWH)	0	644,305	25,868,844
18. Net Electrical Energy (MWH)	0	611,521	24,536,605
19. Unit Service Factor	.0	9.3	58.1
20. Unit Availability Factor	.0	9.3	58.1
21. Unit Capacity Factor (MDC Net)	.0	9.0	54.2
22. Unit Capacity Factor (DER Net)	.0	8.5	51.1
23. Unit Forced Outage Rate	.0	.0	21.0

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 05/05/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
79-10	02/04/79	S	744.0	C	4				CONTINUATION OF SHUTDOWN FOR REFUELING AND STEAM GENERATOR REPLACEMENT WHICH BEGAN ON 2/4/79.

SUMMARY

THE UNIT REMAINED SHUTDOWN THROUGHOUT THE REPORT PERIOD FOR STEAM GENERATOR REPLACEMENT.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

THREE MILE ISLAND 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Pennsylvania/Dauphin 10 miles SE of Harrisburg
2.	Electric Reliability Council	MAAC
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2535
5.	Design Electrical Rating (MWE Net)	819
6.	Date of Initial Criticality	June 5, 1974
7.	Date of Commercial Operation	September 2, 1974
8.	Condenser Cooling Method	Closed
9.	Condenser Cooling Water Source	Susquehanna River

Utility & Contractor Information

10.	Licensee	Metropolitan Edison Company
11.	Corporate Address	P. O. Box 542 Reading, Pennsylvania
12.	Corporate Contact	R. C. Arnold, Sr. Vice-President 260 Cherry Hill Rd. Parsippany, NJ 07054
13.	Architect/Engineer	Gilbert Associates, Inc.
14.	Nuclear Steam Supply System	B&W
15.	Constructor	UE&C

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Senior Inspector	R. R. Keimig
19.	IE Principal Inspector	D. R. Haverkamp
20.	Licensing Project Manager	D. C. DiIanni
21.	Docket Number	50-289
22.	License Number/Date of Issue	DPR-50, 4/19/74
23.	Public Document Room	Government Publications Section State Library of Pennsylvania Education Building Commonwealth and Walnut Street Harrisburg, Pennsylvania 17126

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

None

FACILITY ITEMS (PLANS AND PROCEDURES):

None

MANAGERIAL ITEMS:

None.

PLANT STATUS:

The plant remains shutdown by NRC order, pending completion of modifications and other actions related to the TMI-2 accident.

+ LAST IE SITE INSPECTION DATE: 11/28/79 - 01/04/80

+ INSPECTION REPORT NO: 79-24

SUMMARY

INSPECTION SUMMARY

+ Oct 23 - Nov 27 (79-21): Special inspection by NRC TMI Site staff of: action on previous inspection findings; plant operations including shift activities as monitored by NRC shift inspectors; and, in-plant health physics including documentation, posting and labeling reviews. The inspection included continuous shift coverage by NRC inspectors. No items of noncompliance were identified.

+ Oct 30-31 (79-22): Routine, unannounced inspection by a region-based inspector (14 hours onsite) of the licensee's activities associated with repairing the Borated Water Storage Tank (BWST) roof. No items of noncompliance were identified.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

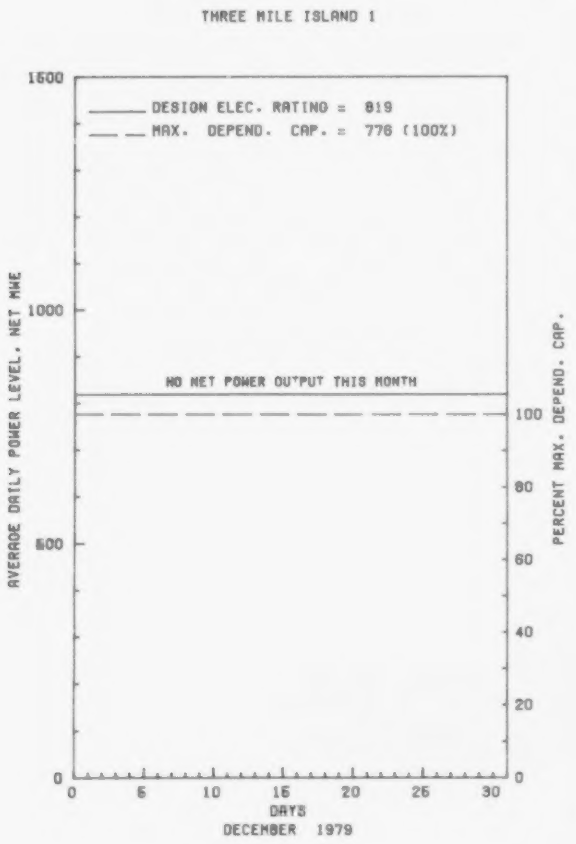
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
NONE				

REVIEWED BY OMPA LOEB: P E Bola DATE: 2/11/80

REVIEWED BY NRR DOR: D C Di DATE: 2/13/80

THREE MILE ISLAND 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-289
2. Utility Contact: D. G. Mitchell (215) 921-6579
3. Licensed Thermal Power (MWt): 2535
4. Nameplate Rating (Gross MWe): 968 x 0.9 = 871
5. Design Electrical Rating (Net MWe): 819
6. Maximum Dependable Capacity (Gross MWe): 840
7. Maximum Dependable Capacity (Net MWe): 776
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	8,760.0	46,729.0
12. Hours Reactor Was Critical	.0	1,128.0	31,731.8
13. Reactor Reserve Shutdown Hours	.0	.0	838.5
14. Hours Generator On-Line	.0	1,128.0	31,180.9
15. Unit Reserve Shutdown Hours	.0	0	.2
16. Gross Thermal Energy (MWH)	0	2,828,448	76,531,071
17. Gross Electrical Energy (MWH)	0	945,996	25,484,330
18. Net Electrical Energy (MWH)	-3,763	848,038	23,840,053
19. Unit Service Factor	.0	12.9	66.7
20. Unit Availability Factor	.0	12.9	66.7
21. Unit Capacity Factor (MDC Net)	.0	12.5	65.7
22. Unit Capacity Factor (DER Net)	.0	11.8	62.3
23. Unit Forced Outage Rate	100.0	85.4	20.7

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: UNKNOWN

UNIT SHUTDOWNS/REDUCTIONS

No	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
1	03/01/79	F	744.0	D	4				REGULATORY RESTRAINT ORDER.

SUMMARY

THE UNIT REMAINS SHUTDOWN DUE TO REGULATORY CONSIDERATION OF THE TMI-2 ACCIDENT

LEGEND

[1] Type:
F–Forced
S–Scheduled

[2] Reason:
A–Equipment Failure
B–Maintenance or Test
C–Refueling
D–Regulatory Restriction
E–Operator Training & License Examination
F–Administrative
G–Operational Error
H–Other

[3] Method:
1–Manual
2–Manual Scram
3–Automatic Scram
4–Continuations
5–Load Reduction
9–Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

THREE MILE ISLAND 2

NRC POWER RESTRICTIONS:

FACILITY DATA

Facility Description

1. Location (state/county) (nearest population center)	Pennsylvania/Dauphin 10 miles SE of Harrisburg
2. Electric Reliability Council	MAAC
3. Type of Reactor	PWR
4. License Power Level (MWT)	2772
5. Design Electrical Rating (MWE Net)	906
6. Date of Initial Criticality	3/28/78
7. Date of Commercial Operation	12/30/78
8. Condenser Cooling Method	CLOSED
9. Condenser Cooling Water Source	Susquehanna River

Utility & Contractor Information

10. Licensee	Metropolitan Edison Company
11. Corporate Address	P. O. Box 542 Reading, PA 19603
12. Corporate Contact	R. C. Arnold, Sr. Vice-President 260 Cherry Hill Rd. Parsippany, NJ 07054
13. Architect/Engineer	Burns & Roe
14. Nuclear Steam Supply System	B&W
15. Constructor	UE&C

Regulatory Information

16. IE Region responsible	I
17. IE Branch Chief	E. J. Brunner
18. IE Senior Inspector	R. R. Keimig
19. IE Principal Inspector	D. R. Haverkamp (X263)
20. Licensing Project Manager	R. VOLLMER
21. Docket Number	50-320
22. License Number/Date of Issue	DPR-73/2; 2/8/78
23. Public Document Room	Government Publications Section State Library of Pennsylvania Education Building Commonwealth & Walnut Streets Harrisburg, Pennsylvania 17126

INSPECTION STATUS

ENFORCEMENT STATUS

None

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

Recovery operations from the 3/28/79 incident.

FACILITY ITEMS (PLANS AND PROCEDURES):

* None

MANAGERIAL ITEMS:

* None

PLANT STATUS:

No significant information to report.

* LAST IE SITE INSPECTION DATE: 01/04/80

INSPECTION REPORT NO: 79-30

SUMMARY

INSPECTION SUMMARY

* Mar 19-23 (79-06): Routine, unannounced physical protection inspection to include: Security Plan; Security Program Audit; Security Organization; physical barriers for protected areas; physical barriers for vital areas; lighting; access control-identification; authorization and badging; access control-search; access control-escorting; access control-vital areas; alarm stations, detection aids; assessment aids; communications; testing and maintenance; compensatory measures; power supply; response; locks, keys and combinations, and records and reports. The inspection involved 58 hours onsite by two NRC inspectors. One apparent item of noncompliance was identified (access control-vital areas).

* Oct 23 - Nov 27 (79-29): Special inspection by NRC TMI Site staff of: action on previous inspection findings; new and revised procedures; plant operations including shift activities as monitored by NRC shift inspectors; fire drill at radwaste storage area; EPICOR II operating experience and health physics practices; and, in-plant health physics including documentation, posting and labeling reviews. The inspection included continuous shift coverage by NRC inspectors. One item of noncompliance was identified (failure to store and control licensed materials in unrestricted areas).

LICENSE STATUS

By Order for Modification of License dated July 20, 1979, the licensee's authority to operate the facility was suspended and the licensee's authority was limited to maintenance of the facility in the present shutdown cooling mode (44 F.R. 45271).

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-020/ 04L-0	11/01/79	12/21/79	Environmental	Failure to conduct Fish Movement and Population Estimates Monitoring Studies

REVIEWED BY OMPA LOEB: PE Sobe **DATE:** 2/11/80

REVIEWED BY NRR DOR: **DATE:** [illegible]

THREE MILE ISLAND 2

AVERAGE DAILY POWER LEVEL (MWe)

OPERATING STATUS

1. Docket Number: 50-320
2. Utility Data Prepared By: R. A. Lengel (215) 921-6581
3. Licensed Thermal Power (MWt): 2772
4. Nameplate Rating (Gross MWe): 1068 X 0.9 = 961
5. Design Electrical Rating (Net MWe): 906
6. Maximum Dependable Capacity (Gross MWe): 0
7. Maximum Dependable Capacity (Net MWe): 0
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons: None
9. Power Level To Which Restricted, If Any (Net MWe): None
10. Reasons For Restrictions, If Any

This Month Yr

11. Hours In Reporting Period
12. Number Of Hours Reactor Was Critical
13. Reactor Reserve Shutdown Hours
14. Hours Generator On-Line
15. Unit Reserve Sh
16. Gro

(MWH)

19

20. U Availability Factor
21. Unit Capacity Factor (Using MDC Net) 1/
22. Unit Capacity Factor (Using DER Net) 1/
23. Unit Forced Outage Rate
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): None
25. If Shut Down At End Of Report Period, Estimated Date of Startup: N/A

NO DECISION ON FUTURE OPERATION
SHUTDOWN INDEFINITELY

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence

SEE SUMMARY BELOW.

SUMMARY

THE UNIT HAS REMAINED SHUTDOWN SINCE THE 03/28/79 ACCIDENT, WHICH CAUSED EXTENSIVE CORE DAMAGE.

AUTHORITY TO OPERATE FACILITY SUSPENDED BY NRC ORDER DATED JULY 20, 1979,
WHICH REQUIRES LICENSEE TO MAINTAIN FACILITY IN A SHUTDOWN CONDITION.

LEGEND

[1] Type:
F--Forced
S-Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

TROJAN

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Oregon/Columbia 30 miles NW of Portland
2.	Electric Reliability Council	WSCC
3.	Type of Reactor	PWR
4.	Licenses Power Level (MWT)	3411
5.	Design Electrical Rating (MWE Net)	1130
6.	Date of Initial Criticality	December 15, 1975
7.	Date of Commercial Operation	May 20, 1976
8.	Condenser Cooling Method	Natural draft cooling tower
9.	Condenser Cooling Water Source	Columbia River

Utility & Contractor Information

10.	Licensee	Portland General Electric Co.
11.	Corporate Address	121 S. W. Salmon St. Portland, Oregon 97204
12.	Corporate Contact	C. Goodwin, Jr., Asst. V.P.
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Multiple

Regulatory Information

16.	IE Region responsible	V
17.	IE Branch Chief	J. L. Crews
18.	IE Senior Inspector	D. M. Sternberg
19.	IE Principal Inspector (Resident)	M. H. Malmros
20.	Licensing Project Manager	C. M. Trammell
21.	Docket Number	50-344
22.	License Number/Date of Issue	NPF-1, 11/21/75
23.	Public Document Room	Columbia County Courthouse Law Library Circuit Court Room St. Helens, Oregon 97501

INSPECTION STATUS

ENFORCEMENT STATUS

PARAGRAPH 20.103(A)(3) REQUIRES THE LICENSEE TO USE SUITABLE MEASUREMENTS OF CONCENTRATIONS OF RADIOACTIVE MATERIALS IN AIR FOR DETECTING AND EVALUATING AIRBORNE RADIOACTIVITY IN RESTRICTED AREAS WHEN DETERMINING COMPLIANCE WITH THE REQUIREMENTS OF SECTION 20.103, "EXPOSURE OF INDIVIDUALS TO CONCENTRATIONS OF RADIOACTIVE MATERIALS IN AIR IN RESTRICTED AREAS." CONTRARY TO THIS REQUIREMENT, AIR SAMPLES WERE NOT TAKEN WHEN A CONTRACTOR EMPLOYEE PERFORMED WORK INSIDE A STEAM GENERATOR ON OCTOBER 25, 1979 OR WHEN A CONTRACTOR EMPLOYEE(S) EARLIER HAD ENTERED STEAM GENERATORS FOR THE PURPOSE OF SETTING UP THE EDDY CURRENT EQUIPMENT.
(7921 INFRACTION)

PARAGRAPH 2.C(7) OF THE TROJAN FACILITY LICENSE, NRC LICENSE NO. NPF-1, REQUIRES ACTIVITIES TO WHICH A QUALITY ASSURANCE PROGRAM IS APPLICABLE (EXCEPT FOR PIPING SUPPORTS) TO BE CONDUCTED IN ACCORDANCE WITH THE QA PROGRAM FOR OPERATION. SECTION II.B.4 OF TROJAN QA PROCEDURE QAP-11-1 REQUIRES TEST RESULTS TO BE REVIEWED AND SPECIFIES ACTIONS TO BE TAKEN WHEN TEST RESULTS DO NOT MEET ACCEPTANCE CRITERIA. CONTRARY TO THESE REQUIREMENTS, THE TEST RESULTS FOR A PERIODIC OPERATING TEST OF THE "C" SERVICE WATER PUMP PERFORMED ON SEPTEMBER 30, 1979 WERE APPROVED BY RESPONSIBLE REVIEWERS EVEN THOUGH RECORDED PUMP DISCHARGE PRESSURE WAS LESS THAN THE SPECIFIED MINIMUM, AND THE ACTIONS SPECIFIED FOR UNACCEPTABLE TEST RESULTS WERE NOT TAKEN.
(7922 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST (CITE RO REPORT NUMBER, IF APP.)

SYSTEMS AND COMPONENT PROBLEMS: -

- Pipe support and concrete block wall modifications were completed on December 21, 1979.

FACILITY ITEMS (PLANS AND PROCEDURES):

- The Hearing Board removed its shutdown order following a public hearing conducted in Portland on December 28, 1979.

MANAGERIAL ITEMS:

None.

PLANT STATUS

- The reactor was started up on December 31, 1979 in preparation for return to power operation.

LAST IE SITE INSPECTION DATE: 12/03-31/79+

INSPECTION REPORT NO: 50-344/79-24+

SUMMARY

INSPECTION SUMMARY:

- Inspection on November 26-December 12, 1979 (Report No. 50-344/79-20)
  Areas Inspected: Included a review of: security plan; security program audit; security organization; protected area physical barriers; vital area physical barriers; lighting; access control (identification, authorization and badging); access control (search); access control (escorting); access control (vital areas); alarm stations; assessment aids; communications; testing and maintenance; response; locks, keys and combinations; followup on previous items of noncompliance/deviations; followup on unresolved items; followup on IE Bulletin 79-16. The inspection involved 116 inspector hours onsite by three NRC inspectors.

  Results: The licensee was found to be in compliance with NRC requirements within the 16 areas examined during this inspection with the following exceptions: (1) Access controls (identification, authorization and badging) - Badge for former site employee not on current access list was available for issue - Infraction. (2) Access control (identification, authorization and badging) - Former contractor employee with no current work related need for access to site was on access list and access badge was on file - Infraction. (3) Access controls (identification, authorization and badging) - Failure of guard to require pocket-carried identification - Infraction. (4) Access control (vital area) - Contract employees not authorized to allow visitor access to the site or vital areas did allow the site and vital area access - Violation.

- Inspection on October 25-26 & November 5-7, 1979 (Report No. 50-344/79-21)
  Areas Inspected: Emergency drill; possible overexposure; licensee investigation of steam generator exposures; radiation protection training; surveys; radiation work permits; organization; QA report of rad waste and shipping audit; detection of tritium in recreation lake; and tour of containment. The inspection involved 52 inspector hours onsite by two NRC inspectors.

  Results: Of the ten areas inspected, no items of noncompliance or deviations were found in nine areas; one item of noncompliance was found in one area (Infraction - Failure to take an air sample while persons were working in the steam generators).

- Inspection on November 6-8, 1979 (Report No. 50-344/79-22)
  Areas Inspected: Routine, unannounced inspection of employee training; response to IE Bulletins and Circulars; and followup on previous inspection items; also included independent inspection effort. The inspection involved 34 inspector hours onsite by two NRC inspectors.

  Results: Of the two areas inspected, one noncompliance item was identified in one area (Infraction - Failure to take followup actions on unacceptable surveillance test results).

- Inspection on November 1-30, 1979 (Report No. 50-344/79-23)
  Routine inspections of plant operations, maintenance, physical security, calibration, environmental and radiological protection, and followup on a licensee event report. The inspection involved 78 inspector hours by the NRC Resident Inspector.

  Results: One item of noncompliance was identified (Infraction - Failure to follow quality assurance procedure for calibration of installed plant instrumentation). No deviations were identified.

- Inspection on December 3-31, 1979 (Report No. 50-344/79-24)
  Report being prepared; to be reported next month.

- Inspection on November 26-28 & December 4, 1979 (Report No. 50-344/79-25)
  Areas Inspected: Special, announced inspection of the circumstances surrounding and corrective actions taken by the licensee as a consequence of Licensee Event Report (LER 79-15). The inspection involved 63 inspector hours by three NRC inspectors.

  Results: No items of noncompliance or deviations were identified.

REPORTS RECEIVED FROM LICENSEE

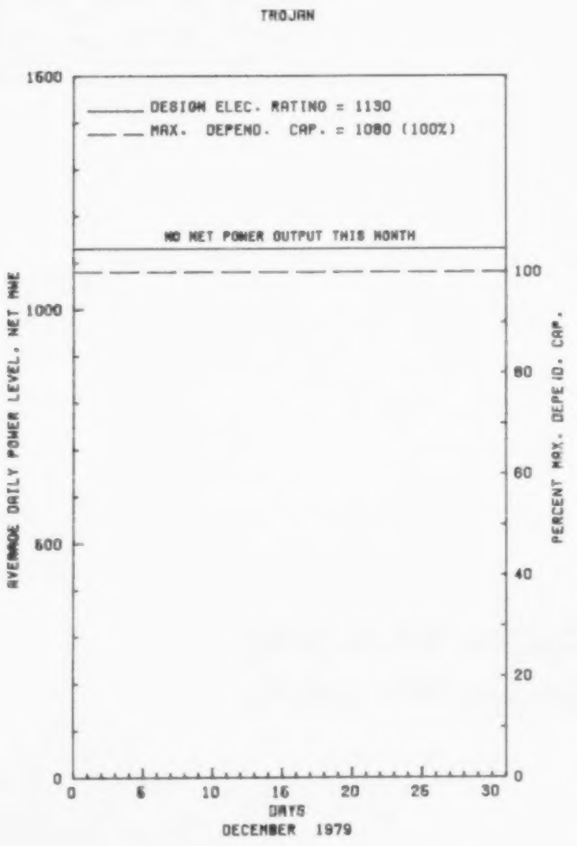
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
--	10-18-79	10-24-79	Environmental	PH of effluent above limit in technical specifications.
--	09-06-79	12-03-79	Special	Inadvertent actuation of ECCS.

REVIEWED BY OMPA LOEB: T. C. Cintula **DATE:** 2-11-80

REVIEWED BY NRR DOR: CM Trammell **DATE:** 2-13-80

TROJAN

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-344
2. Utility Contact: G. G. Bair (503) 556-3713 x234
3. Licensed Thermal Power (MWt): 3411
4. Nameplate Rating (Gross MWe): 1280 x 0.95 = 1216
5. Design Electrical Rating (Net MWe): 1130
6. Maximum Dependable Capacity (Gross MWe): 1122
7. Maximum Dependable Capacity (Net MWe): 1080
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	8,760.0	29,208.0
12. Hours Reactor Was Critical	14.0	5,175.6	16,298.0
13. Reactor Reserve Shutdown Hours	.0	8.9	2,171.8
14. Hours Generator On-Line	3.6	5,087.4	15,627.8
15. Unit Reserve Shutdown Hours	.0	.5	1,508.7
16. Gross Thermal Energy (MWH)	7,662	16,956,391	48,895,610
17. Gross Electrical Energy (MWH)	1,420	5,534,645	15,985,415
18. Net Electrical Energy (MWH)	-7,489	5,266,720	15,033,319
19. Unit Service Factor	.5	58.1	53.5
20. Unit Availability Factor	.5	58.1	58.7
21. Unit Capacity Factor (MDC Net)	.0	55.7	47.7
22. Unit Capacity Factor (DER Net)	.0	53.2	45.5
23. Unit Forced Outage Rate	.0	2.0	30.6

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUELING, APRIL 1980 - 90 DAYS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: 01/01/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
79-13	10/12/79	S	748.4	D	4				SCHEDULED OUTAGE OCCURRED TO PERMIT NRC REQUIRED INSPECTION OF HANGERS AND RESTRAINTS AND TO PLUG SMALL STEAM GENERATOR TUBE LEAKS IN THE "A" AND "D" STEAM GENERATORS. OUTAGE SUBSEQUENTLY EXTENDED DUE TO LICENSING HEARING REGARDING THE PROGRESS TO RESOLVING THE CONTROL BUILDING SEISMIC ISSUE.
79-14	12/31/79	F	.0	H	5				TROUBLE WITH MAINTAINING CONDENSER VACUUM WHEN HOGGER AIR EJECTION SECURED; REQUIRED LOAD REDUCTION FROM 57% TO 30% LOAD AND AGAIN FROM 90% TO 30% LOAD.
79-15	12/31/79	F	.0	H	5				TROUBLE WITH MAINTAINING CONDENSER VACUUM WHEN HOGGER AIR EJECTION SECURED; REQUIRED LOAD REDUCTION FROM 57% TO 30% LOAD AND AGAIN FROM 90% TO 30% LOAD.

SUMMARY

THE UNIT REMAINED SHUTDOWN UNTIL 12/31/79 PENDING CONTROL BUILDING SEISMIC REDESIGN. INTERIM OPERATION BEGAN AT THE END OF THE REPORT PERIOD, FOLLOWING ASLB AND NRC APPROVAL.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

TURKEY POINT 3

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1. Location (state/county) (nearest population center)	Florida/Dade 25 miles S of Miami
2. Electric Reliability Council	Southeastern
3. Type of Reactor	PWR
4. Power Level (MWT)	2200
5. Design Electrical Rating (MWE Net)	693
6. Date of Initial Criticality	October 20, 1972
7. Date of Commercial Operation	December 14, 1972
8. Condenser Cooling Method	Recirculating Canal System
9. Condenser Cooling Water Source	Biscayne Bay

Utility & Contractor Information

10. Licensee	Florida Power and Light Company
11. Corporate Address	9250 West Flagler Street P. O. Box 013100 Miami, Florida 33101
12. Corporate Contact	R. E. Uhrig, V.P. Nuclear Affairs
13. Architect/Engineer	Bechtel
14. Nuclear Steam Supply System	Westinghouse
15. Constructor	Bechtel

Regulatory Information

16. IE Region responsible	II
17. IE Branch Chief	R. C. Lewis
18. IE Senior Inspector	E. H. Verdery
19. IE Principal Inspector	R.J. Vogt-Lowell
20. Licensing Project Manager	M. Grotenhuis
21. Docket Number	50-250
22. License Number/Date of Issue	DPR-31, 7/19/72
23. Public Document Room	Environmental and Urban Affairs Library Florida International University Miami, Florida 33199

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

SHUTDOWN ON 12/1/79 FOR EXTENDED REFUELING OUTAGE. IEB 79-02, 79-13 AND 79-14 INSPECTIONS TO BE PERFORMED DURING OUTAGE. RESTART IS SCHEDULED FOR 1/25/80.

+ LAST IE SITE INSPECTION DATE: DECEMBER 1-20, 1979

+ INSPECTION REPORT NO: 50-250/79-41

SUMMARY

INSPECTION SUMMARY

+ INSPECTION NOVEMBER 11-16 (79-33): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 13 INSPECTOR-HOURS ON SITE IN THE AREAS OF LIQUID, GASEOUS AND SOLID RADWASTES, RECORDS AND REPORTS OF RADIOACTIVE EFFLUENTS, PROCEDURES FOR CONTROLLING EFFLUENT RELEASES, RESPIRATORY PROTECTION PROGRAM, FILTRATION SYSTEMS, AND ALARA CONSIDERATION. OF THE NINE AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION NOVEMBER 27-30 (79-35): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 22 INSPECTOR-HOURS ON SITE AND AT THE GENERAL OFFICE IN REVIEW OF PROCEDURES AND CONTROLS ADOPTED BY FLORIDA POWER AND LIGHT (FP&L) TO IMPLEMENT THE REQUIREMENTS OF 10 CFR PART 21 AND 10 CFR 50.55(E). OF THE TWO AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN ONE AREA; ONE ITEM OF NONCOMPLIANCE WAS FOUND IN ONE AREA (INFRACTION - 10 CFR PART 21 PROCEDURES).

REPORTS RECEIVED FROM LICENSEE

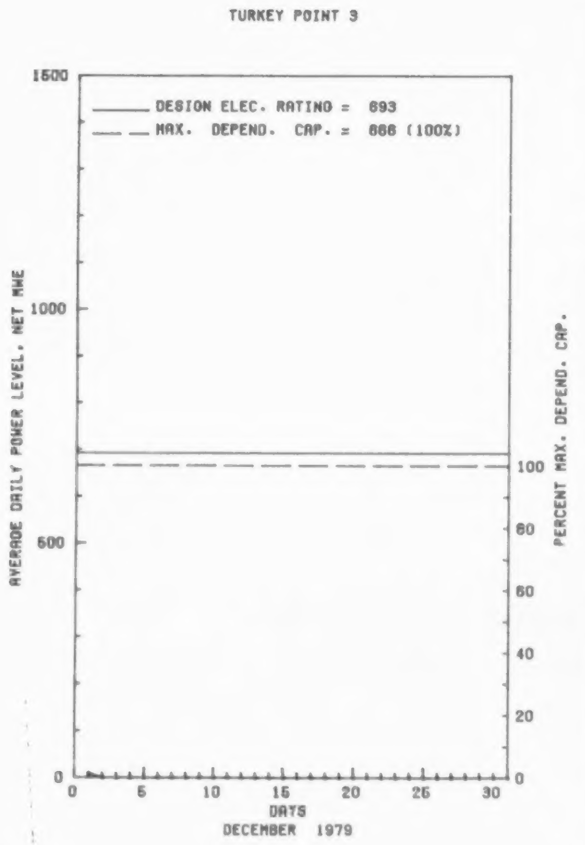
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-034/ 03L-0	11/05/79	12/05/79	30 DAY LER	AUXILIARY FEED PUMP "A" FAILED TO START DURING TEST.

REVIEWED BY OMPA LOEB: T.C. CINTULA DATE: 2•11•80

REVIEWED BY NRR DOR: M Grotenhuis DATE: 2·13·80

TURKEY POINT 3

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

ITEM 21 CALCULATED BY USING A WEIGHTED AVERAGE

1. Docket Number: 50-250
2. Utility Contact: V. T. Chilson (305) 552-3824
3. Licensed Thermal Power (MWt): 2200
4. Nameplate Rating (Gross MWe): 894 x 0.85 = 760
5. Design Electrical Rating (Net MWe): 693
6. Maximum Dependable Capacity (Gross MWe): 700
7. Maximum Dependable Capacity (Net MWe): 666
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	8,760.0	62,001.6
12. Hours Reactor Was Critical	2.6	4,697.0	47,269.4
13. Reactor Reserve Shutdown Hours	.0	18.7	213.3
14. Hours Generator On-Line	2.4	4,513.0	45,690.2
15. Unit Reserve Shutdown Hours	.0	24.5	121.8
16. Gross Thermal Energy (MWH)	2,229	9,701,686	91,468,634
17. Gross Electrical Energy (MWH)	585	3,046,570	29,081,000
18. Net Electrical Energy (MWH)	-1,405	2,874,917	27,522,094
19. Unit Service Factor	.3	51.5	73.7
20. Unit Availability Factor	.3	51.8	73.9
21. Unit Capacity Factor (MDC Net)	.0	49.3	67.2
22. Unit Capacity Factor (DER Net)	.0	47.4	64.1
23. Unit Forced Outage Rate	.0	3.1	2.7

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 01/26/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
23	12/01/79	S	741.6	C	1		RC	FUELXX	UNIT NO. 3 WAS REMOVED FROM SERVICE FOR SCHEDULED REFUELING, MAINTENANCE, AND INSPECTIONS.

SUMMARY

THE UNIT WAS REMOVED FROM SERVICE ON 12/01/79 FOR REFUELING.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

best copy available

TURKEY POINT 4

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Florida/Dade 25 miles S of Miami
2.	Electric Reliability Council	Southeastern
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2200
5.	Design Electrical Rating (MWE Net)	693
6.	Date of Initial Criticality	June 11, 1973
7.	Date of Commercial Operation	September 7, 1973
8.	Condenser Cooling Method	Recirculating Canal System
9.	Condenser Cooling Water Source	Biscayne Bay

Utility & Contractor Information

10.	Licensee	Florida Power and Light Company
11.	Corporate Address	9250 West Flagler Street P. O. Box 013100 Miami, Florida 33101
12.	Corporate Contact	R. E. Uhrig, V. P. Nuclear Affairs
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. Lewis
18.	IE Senior Inspector	E.H. Verdery
19.	IE Principal Inspector	R.J. Vogt-Lowell
20.	Licensing Project Manager	M. Grotenhuis
21.	Docket Number	50-251
22.	License Number/Date of Issue	DPR-41, 4/10/73
23.	Public Document Room	Environmental and Urban Affairs Library Florida International University Miami, Florida 33199

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

UNIT OPERATING AT 100% POWER.

+ LAST IE SITE INSPECTION DATE: DECEMBER 1-20, 1979

+ INSPECTION REPORT NO: 50-251/79-41

SUMMARY

INSPECTION SUMMARY

+ INSPECTION NOVEMBER 11-16 (79-33): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 14 INSPECTOR-HOURS ON SITE IN THE AREAS OF LIQUID, GASEOUS AND SOLID RADWASTES, RECORDS AND REPORTS OF RADIOACTIVE EFFLUENTS, PROCEDURES FOR CONTROLLING EFFLUENT RELEASES, RESPIRATORY PROTECTION PROGRAM, FILTRATION SYSTEMS, AND ALARA CONSIDERATION. OF THE NINE AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION NOVEMBER 27-30 (79-35): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 22 INSPECTOR-HOURS ON SITE AND AT THE GENERAL OFFICE IN REVIEW OF PROCEDURES AND CONTROLS ADOPTED BY FLORIDA POWER AND LIGHT (FP&L) TO IMPLEMENT THE REQUIREMENTS OF 10 CFR PART 21 AND 10 CFR 50.55(E). OF THE TWO AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN ONE AREA; ONE ITEM OF NONCOMPLIANCE WAS FOUND IN ONE AREA (INFRACTION - 10 CFR PART 21 PROCEDURES).

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

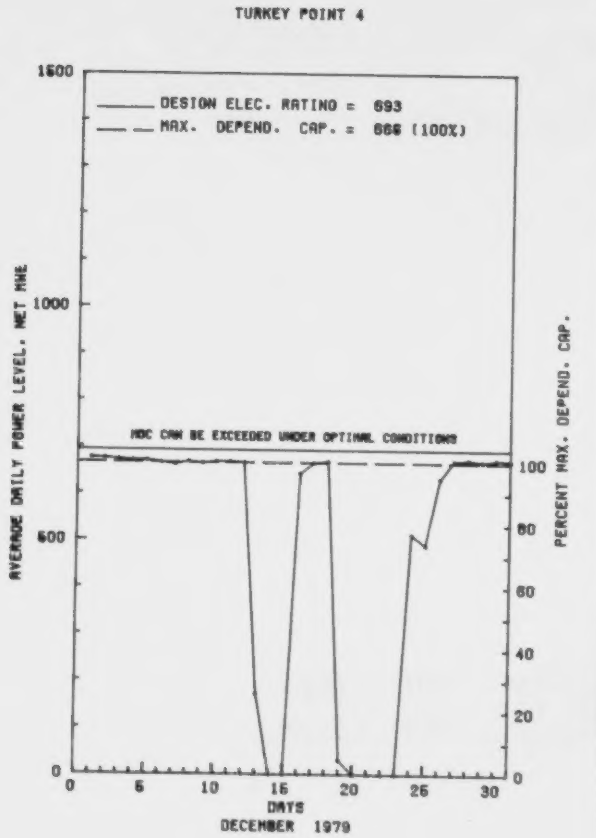
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
		NONE		

REVIEWED BY OMPA LOEB: T.C. Cintula DATE: 2•11•80

REVIEWED BY NRR DOR: M Grotenhuis DATE: 2.13.80

TURKEY POINT 4

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

ITEM 21 CALCULATED USING A WEIGHTED AVERAGE

1. Docket Number: 50-251
2. Utility Contact: V. T. Chilson (305) 552-3824
3. Licensed Thermal Power (MWt): 2200
4. Nameplate Rating (Gross MWe): 894 x 0.85 = 760
5. Design Electrical Rating (Net MWe): 693
6. Maximum Dependable Capacity (Gross MWe): 700
7. Maximum Dependable Capacity (Net MWe): 666
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	8,760.0	55,729.0
12. Hours Reactor Was Critical	567.5	6,480.1	40,827.8
13. Reactor Reserve Shutdown Hours	.0	15.5	154.3
14. Hours Generator On-Line	558.9	6,363.4	39,182.5
15. Unit Reserve Shutdown Hours	.0	18.9	18.9
16. Gross Thermal Energy (MWH)	1,205,087	13,079,113	81,218,980
17. Gross Electrical Energy (MWH)	384,290	4,067,753	25,903,478
18. Net Electrical Energy (MWH)	363,521	3,845,291	24,529,996
19. Unit Service Factor	75.1	72.6	70.3
20. Unit Availability Factor	75.1	72.9	70.3
21. Unit Capacity Factor (MDC Net)	73.4	65.9	66.5
22. Unit Capacity Factor (DER Net)	70.5	63.3	63.5
23. Unit Forced Outage Rate	24.9	5.7	3.3

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): SCHED. MAINT. & INSPEC. - APRIL 27 - MAY 31, 1980.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
24	12/13/79	F	62.1	A	1		HB	PIPEXX	UNIT WAS REMOVED FROM SERVICE TO REPAIR STEAM LEAK ON STEAM GENERATOR NO. 4B STEAM FLOW SENSING LINE THAT COULD NOT BE ISOLATED. CORRECTIVE ACTIONS INCLUDED REPAIRS BY WELDING.
25	12/15/79	F	.6	A	9		EA	CKTBRK	UNIT WAS REMOVED FROM SERVICE TO REPAIR GENERATOR DISCONNECT SWITCHES. CORRECTIVE ACTIONS INCLUDED EXERCISING THE DISCONNECT SWITCHES.
26	12/19/79	F	122.4	A	3		EB	ELECON	UNIT WAS TRIPPED BY REACTOR PROTECTION SYSTEM WHEN REACTOR COOLANT PUMP NO. 4A MOTOR OVERCURRENT RELAYS ACTUATED AND TRIPPED PUMP WITH REACTOR POWER LEVEL GREATER THAN 45% R.P. CORRECTIVE ACTIONS INCLUDED REPAIRING MOTOR LEADS AT MOTOR TERMINAL BOX.
27	12/25/79	F	.0	A	5		HC	HTEXCH	LOAD REDUCTION TO REPAIR CONDENSER TUBE LEAK.

SUMMARY

OPERATION WAS AT NEAR FULL POWER DURING THE REPORT PERIOD EXCEPT FOR OUTAGES ON 12/13/79 TO REPAIR A STEAM LEAK THAT COULD NOT BE ISOLATED, ON 12/15/79 TO REPAIR GENERATOR DISCONNECT SWITCHES, AND ON 12/19/79 FOLLOWING A UNIT TRIP BY THE REACTOR PROTECTION SYSTEM. A BRIEF LOAD REDUCTION OCCURRED ON 12/25/79 TO REPAIR A CONDENSER TUBE LEAK.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

VERMONT YANKEE

FACILITY DATA

Facility Description

1. Location (state/county) (nearest population center)	Vermont/Windham 5 miles S of Brattleboro
2. Electric Reliability Council	NPCC
3. Type of Reactor	BWR
4. Power Level (MWT)	1593
5. Design Electrical Rating (MWE Net)	514
6. Date of Initial Criticality	March 24, 1972
7. Date of Commercial Operation	November 30, 1972
8. Condenser Cooling Method	Closed Cycle Cooling Towers
9. Condenser Cooling Water Source	Connecticut River

Utility & Contractor Information

10. Licensee	Vermont Yankee Nuclear Power Corp.
11. Corporate Address	20 Turnpike Road Westboro, Massachusetts 01581
12. Corporate Contact	D. W. Edwards Assistant to the Vice President
13. Architect/Engineer	Ebasco
14. Nuclear Steam Supply System	GE
15. Constructor	Ebasco

Regulatory Information

16. IE Region responsible	I
17. IE Branch Chief	E. Brunner
18. IE Senior Inspector	E. McCabe
19. IE Principal Inspector	T. F. Stetka
20. Licensing Project Manager	V. L. Rooney
21. Docket Number	50-271
22. License Number/Date of Issue	DPR-28, 3/21/72
23. Public Document Room	Brooks Memorial Library 224 Main Street Brattleboro, Vermont 05301

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO TS 3.5.C.2 & 4.5.C.2, RHRSW PUMP WAS INTENTIONALLY MADE INOPERABLE WITHOUT INSTITUTING VERIFICATION OF THE OPERABILITY OF THE REDUNDANT RHRSW SUBSYSTEM.
(7917 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

- The unit was shutdown on January 5 when a packing leak developed in an RHR system isolation valve inside the drywell. No release of radioactive material occurred. Power operation resumed on January 6.

FACILITY ITEMS (PLANS AND PROCEDURES):

- A one week outage is scheduled to begin on February 16 to complete system modifications necessitated by NUREG 0578.

MANAGERIAL ITEMS:

None

PLANT STATUS:

- Operating at 92 percent power, with escalation to full power in progress. SJAE offgas rate is 2650 microcuries per second. Stack release rate is below 100 microcuries per second.

- LAST IE SITE INSPECTION DATE: 01/10/80
- INSPECTION REPORT NO: 80-01

SUMMARY

INSPECTION SUMMARY

- Oct 16-19, 22-26 & 29-31 (79-18): Routine, unannounced inspection of QA Program implementation including: QA Program Changes; Organization; Design changes/Modifications; Document Control including Drawings; and licensee action on previous inspection findings. The inspection involved 130 hours onsite and 40 hours at the corporate offices by two region-based inspectors. No items of noncompliance were identified.
- Oct 29-30 (79-19): Routine, unannounced, onsite inspection (19 hours) by a region-based inspector of the containment integrated leak rate test, backshift operations, previous inspection items and system seismic design. No items of noncompliance were identified.
- Nov 28-30 (79-21): Routine, unannounced inspection of plant operations, including facility records and inspection tours of accessible plant areas; licensee action on previous inspection items; followup on IE Bulletins and Circulars; licensee review of Temporary Procedure Changes; Licensee Event Reports and Monthly Operating Reports; and, licensee action on Small Break LOCA procedures. The inspection involved 40 hours onsite by two region-based inspectors. No items of noncompliance were identified.
- Dec 5-6 (79-22): Special, unannounced inspection by a region-based inspector of the licensee's response to Bulletin 79-19, including: regulatory requirements, burial site requirements, procedures, training, audits, records of shipments, onsite observations, and discrepant shipping reports. No items of noncompliance or deviations were disclosed.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

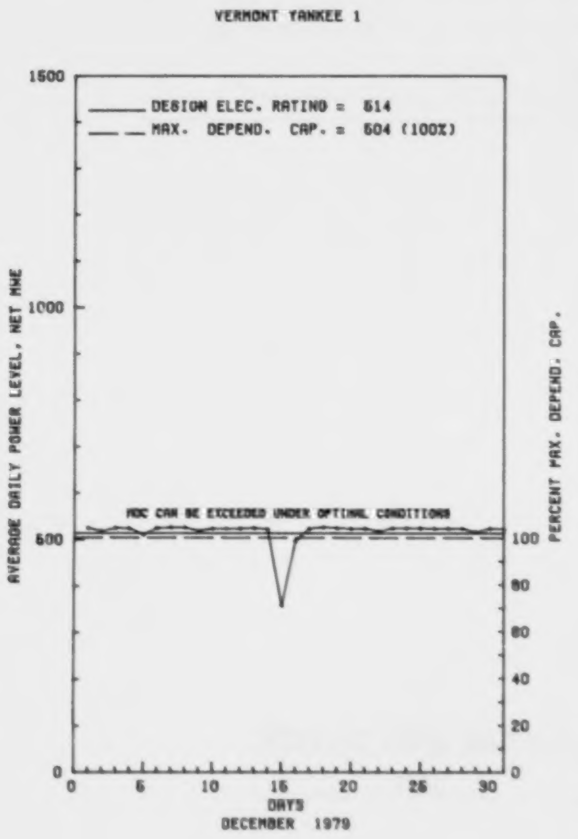
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-029/03L	11/04/79	12/04/79	30 Day LER	During routine surveillance testing, MSIVs 80B & 86C were found to close in 2.5 seconds (the minimum acceptance time is 2 seconds)
79-030/03L	11/10/79	12/10/79	30 Day LER	Torus water level indications drifted high due to draining of common reference leg
79-031/03L	11/10/79	12/14/79	30 Day LER	Three hangers on safety-related systems inadvertently left inoperative during plant startup

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 2-1-80

REVIEWED BY NRR DOR: **DATE:** 2/13/80

VERMONT YANKEE

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-271
2. Utility Contact: R. M. Sjogren (617) 366-9011 X2281
3. Licensed Thermal Power (MWt): 1593
4. Nameplate Rating (Gross MWe): 626 x 0.9 = 563
5. Design Electrical Rating (Net MWe): 514
6. Maximum Dependable Capacity (Gross MWe): 535
7. Maximum Dependable Capacity (Net MWe): 504
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	8,760.0	63,794.0
12. Hours Reactor Was Critical	744.0	7,268.6	51,158.6
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	744.0	7,194.8	49,327.8
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,165,003	10,837,793	69,224,611
17. Gross Electrical Energy (MWH)	400,506	3,630,202	22,969,670
18. Net Electrical Energy (MWH)	384,602	3,448,842	21,768,365
19. Unit Service Factor	100.0	82.1	77.3
20. Unit Availability Factor	100.0	82.1	77.3
21. Unit Capacity Factor (MDC Net)	102.6	78.1	67.7
22. Unit Capacity Factor (DER Net)	100.6	76.6	66.4
23. Unit Forced Outage Rate	.0	1.6	6.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
79-19	12/05/79	F	.0	A	5		RA	CONROD	CONTROL ROD 14-31 SCRAMMED WHILE TROUBLE SHOOTING A PROBLEM WITH ITS SCRAM VALVE.
79-20	12/15/79	F	.0	A	5	79-35/3L	CD	VALVOP	MSIV 80A INOPERABLE DUE TO MECHANICAL BINDING OF THE GUIDE RODS ON THE VALVE OPERATOR. THIS CONDITION WAS REPAIRED AS NECESSARY.

SUMMARY

OPERATION WAS AT NEAR FULL POWER DURING THE REPORT PERIOD, WITH BRIEF REDUCTIONS ON 12/05/79 WHEN A CONTROL ROD SCRAMMED WHILE TROUBLESHOOTING A PROBLEM WITH ITS SCRAM VALVE, AND ON 12/15/79 WHEN A MAIN STEAM ISOLATION VALVE BECAME INOPERABLE FROM MECHANICAL BINDING OF THE GUIDE RODS ON THE VALVE OPERATOR.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

YANKEE ROWE

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Massachusetts/Franklin 25 miles NE of Pittsfield
2.	Electric Reliability Council	NPCC
3.	Type of Reactor	PWR
4.	Power Level (MWT)	600
5.	Design Electrical Rating (MWE Net)	175
6.	Date of Initial Criticality	August 19, 1960
7.	Date of Commercial Operation	July 1961
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Deerfield River

Utility & Contractor Information

10.	Licensee	Yankee Atomic Electric Company
11.	Corporate Address	20 Turnpike Road Westboro, Massachusetts 01581
12.	Corporate Contact	D. W. Edwards, Asst. to V.P.
13.	Architect/Engineer	Stone & Webster
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Senior Inspector	R. R. Keimig
19.	IE Principal Inspector	W. J. Lazarus
20.	Licensing Project Manager	A. Burger
21.	Docket Number	50-29
22.	License Number/Date of Issue	DPR-3, 12/24/63
23.	Public Document Room	Greenfield Community College 1 College Drive Greenfield, Massachusetts 01301

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

♦ None

FACILITY ITEMS (PLANS AND PROCEDURES):

♦ A three week shutdown is scheduled to start January 19 to install voltage regulators on incoming power supplies and to implement Three Mile Island "Short Term Lessons Learned" requirements.

MANAGERIAL ITEMS:

None

PLANT STATUS:

♦ Routine operation.

♦ LAST IE SITE INSPECTION DATE: 12/19/79

♦ INSPECTION REPORT NO: 79-16

SUMMARY

INSPECTION SUMMARY

♦ Nov 1-2 (79-12): Routine, unannounced inspection of the quality related activities associated with the steam generator feedwater pipe replacement. The inspection involved 12 hours onsite by one region-based inspector. No items of noncompliance were identified.

♦ Oct 16-19, 22-26 & 29-31 (79-13): Routine, unannounced inspection of QA Program implementation including: Program changes; QA/QC Organization; Design Changes/Modifications; Document Control, including Drawings; and licensee action on previous inspection findings. The inspection involved 35 hours onsite and 40 hours at the corporate offices by two region-based inspectors. No items of noncompliance were identified.

♦ Nov 7 (79-14): Special unannounced inspection of licensee corrective action for the potential degradation of safeguards bus voltage reported in Licensee Event Report 79-26. The inspection involved 5 hours onsite by one NRC region-based inspector. No items of noncompliance were identified.

REPORTS RECEIVED FROM LICENSEE

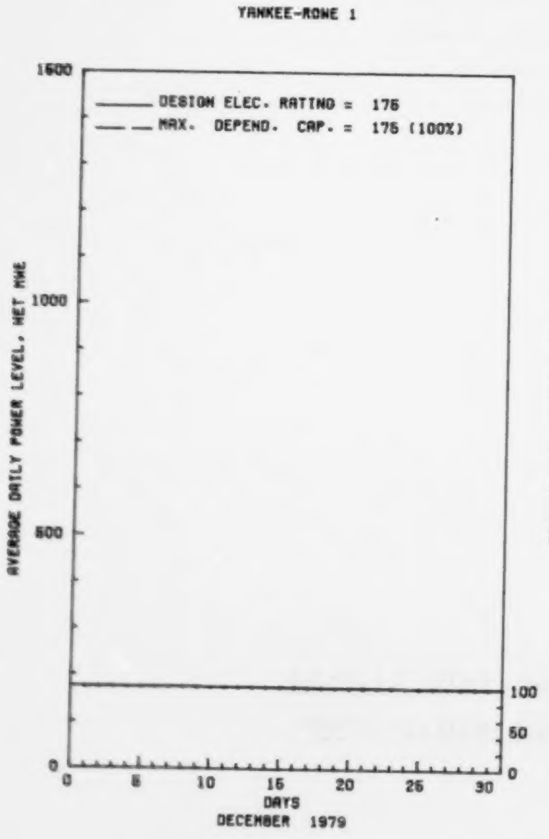
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-027/03L	11/05/79	12/04/79	30 Day LER	HV-SOV-1 & 2 failed Type C LRT
79-028/03L	11/05/79	12/05/79	30 Day LER	Missed surveillance test (timing test on HV-SOV-1&2)
79-029/03L	11/06/79	12/06/79	30 Day LER	HV-SOV 1 & 2 failed LRT
79-030/03L	11/09/79	12/07/79	30 Day LER	Safety injection tank time delay relay failure

REVIEWED BY OMPA LOEB: T. C. Cintula DATE: 2-11-80

REVIEWED BY NRR DOR: DATE:

YANKEE ROWE

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

CAPACITY FACTORS
CALCULATED WITH A
WEIGHTED AVERAGE

1. Docket Number: 50-029
2. Utility Contact: R. M. Sjogren (617) 366-9011 X2281
3. Licensed Thermal Power (MWt): 600
4. Nameplate Rating (Gross MWe): 185 x 1.0 = 185
5. Design Electrical Rating (Net MWe): 175
6. Maximum Dependable Capacity (Gross MWe): 180
7. Maximum Dependable Capacity (Net MWe): 175
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	8,760.0	167,637.0
12. Hours Reactor Was Critical	744.0	7,173.7	138,365.6
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	744.0	7,149.5	134,027.4
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	435,186	4,174,777	71,748,322
17. Gross Electrical Energy (MWH)	138,285	1,311,204	22,072,153
18. Net Electrical Energy (MWH)	129,919	1,232,264	20,666,144
19. Unit Service Factor	100.0	81.6	80.0
20. Unit Availability Factor	100.0	81.6	80.0
21. Unit Capacity Factor (MDC Net)	99.8	80.4	73.1
22. Unit Capacity Factor (DER Net)	99.8	80.4	73.1
23. Unit Forced Outage Rate	.0	2.4	1.6

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
TMI MODS & INSTALL VOLT. REGS. - 01/19/80 - 3 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	NONE								

SUMMARY

OPERATION WAS AT NEAR FULL POWER THROUGHOUT THE REPORT PERIOD, WITH A BRIEF LOAD REDUCTION ON 12/07/79 FOR TURBINE THROTTLE AND CONTROL VALVE EXERCISES.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

ZION 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Illinois/Lake 40 miles N of Chicago
2.	Electric Reliability Council	Main
3.	Type of Reactor	PWR
4.	Power Level (MWT)	3250
5.	Design Electrical Rating (MWE Net)	1040
6.	Date of Initial Criticality	June 19, 1973
7.	Date of Commercial Operation	December 31, 1973
8.	Condenser Cooling Method	Straight Through
9.	Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10.	Licensee	Commonwealth Edison Co.
11.	Corporate Address	P. O. Box 767 Chicago, Illinois
12.	Corporate Contact	D. L. Peoples
13.	Architect/Engineer	Sargent & Lundy
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Various

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Senior Inspector	R. Spessard
19.	IE Resident Inspector.	J.E. Kohler
20.	Licensing Project Manager	E. Reeves
21.	Docket Number	50-295
22.	License Number/Date of Issue	DPR-39 issued 4/6/73
23.	Public Document Room	Zion-Benton Public Library 2600 Emmans Avenue Zion, Illinois 60099

INSPECTION STATUS

ENFORCEMENT STATUS

TECHNICAL SPECIFICATION 6.2.A.11 REQUIRES ADHERENCE TO FIRE PROTECTION PROGRAM IMPLEMENTATION PROCEDURES. CONTRARY TO THE ABOVE, ON OCTOBER 31, 1979, WELDING TOOK PLACE IN THE UNIT 1 TURBINE BUILDING WITHOUT A PORTABLE FIRE EXTINGUISHER AT THE JOB SITE, AS SPECIFIED BY THE APPROVED WELDING AUTHORIZATION CARD.
(7924 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

- Portable demineralizers continue to be used for processing radioactive liquids due to operational problems with the installed cement solidification system. (79-25)

FACILITY ITEMS(PLANS AND PROCEDURES):

- Radwaste shipments were resumed to Barnwell, South Carolina on November 20, 1979 following rescission of a shipping ban imposed by the State of South Carolina. (79-25)

MANAGERIAL ITEMS:

- The licensee has committed to review/revise the procedures and training program for processing, packaging, transfer, and transport of radioactive materials by early 1980. (79-25)

PLANT STATUS

- The unit was shut down on October 6, 1979 to begin a scheduled 8-week refueling/maintenance outage. The outage schedule has been extended by about 7 weeks due to the system and component problems discussed above. The tentative startup date is January 19, 1980.

*LAST IE SITE INSPECTION DATE: November 19-20, 1979

- INSPECTION REPORT NO: 79-25

SUMMARY

INSPECTION SUMMARY

- Inspection on October 1 through November 5, (79-24): Routine, unannounced resident inspection of licensee action on previous inspection findings, maintenance, plant operations including followup of nonroutine events, review and followup of LER's, Unit 1 refueling activities, surveillance and calibration of safety related components required by TS, radioactive waste systems, preparation for Unit 1 refueling, receipt of new fuel for Unit 1, feedwater line cracking on Unit 1, fire protection and diesel generator roller pin inspections. The inspection involved 133 inspector hours onsite by one NRC inspector. One item of noncompliance (failure to follow procedures which implement the fire protection program) was identified.
- Inspection on November 19-20, (79-25): Nonroutine, announced inspection of licensee actions taken in response to IE Bulletin 79-19 "Packaging of Low-Level Radioactive Waste for Transport and Burial" and, as agreed with the State of South Carolina, verification that identified packaging discrepancies were corrected before resumption of radwaste shipments to Barnwell, South Carolina. The inspection involved 28 inspector hours onsite by two NRC inspectors. No items of noncompliance or deviations were identified.

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-79/03L-0	10/23/79	11/09/79	30 Day LER	Auxiliary building Radiation Monitor failed low.
79-85/01T-0	10/26/79	11/05/79	10 Day LER	High pH of effluent from waste water treatment facility.
79-87/03L-0	10/27/79	11/21/79	30 Day LER	Containment vent particulate monitor failed low.
79-90/01T-0	11/20/79	11/20/79	14 Day LER	Non-conservative negative rate flux trip.

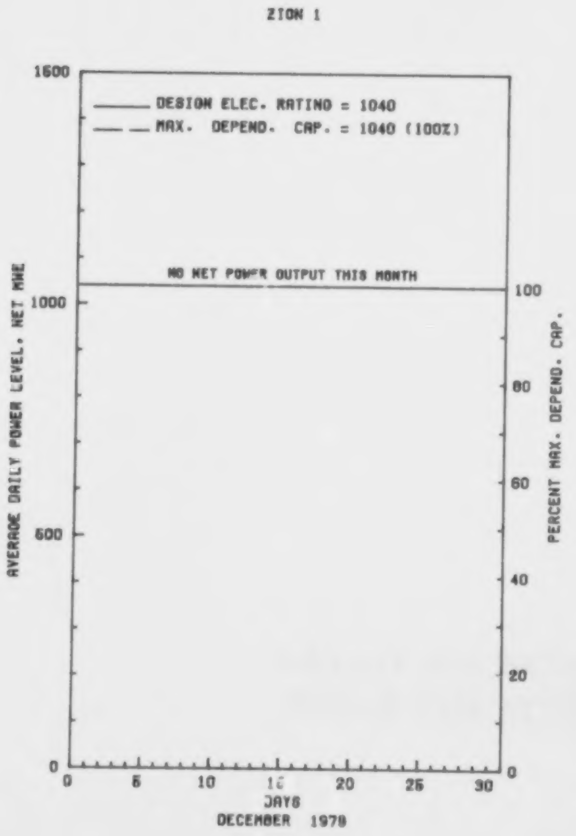
==

REVIEWED BY OMPA LOEB: T.C. Cintula **DATE:** 2·11·80

REVIEWED BY NRR DOR: E.A. Reeves **DATE:** 2-13-80

ZION 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-295
2. Utility Contact: J. Cook (312) 746-2084
3. Licensed Thermal Power (MWt): 3250
4. Nameplate Rating (Gross MWe): 1220 x 0.9 = 1098
5. Design Electrical Rating (Net MWe): 1040
6. Maximum Dependable Capacity (Gross MWe): 1085
7. Maximum Dependable Capacity (Net MWe): 1040
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	8,760.0	52,608.0
12. Hours Reactor Was Critical	.0	6,047.1	37,[illegible]07.7
13. Reactor Reserve Shutdown Hours	.0	.0	2,621.8
14. Hours Generator On-Line	.0	5,965.8	36,087.3
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	17,647,043	99,383,001
17. Gross Electrical Energy (MWH)	0	5,823,455	32,232,050
18. Net Electrical Energy (MWH)	0	5,537,168	30,484,246
19. Unit Service Factor	.0	68.1	68.6
20. Unit Availability Factor	.0	68.1	68.6
21. Unit Capacity Factor (MDC Net)	.0	60.8	55.7
22. Unit Capacity Factor (DER Net)	.0	60.8	55.7
23. Unit Forced Outage Rate	100.0	22.1	13.1

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 01/19/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
26	11/21/79	F	744.0	D	4	79-077	HH	PIPEXX	UNIT DOWN FOR FEEDWATER NOZZLE REPAIR PER NRC BULLETIN #79-13. FOLLOWING REFUELING OUTAGE.

SUMMARY

THE UNIT REMAINED SHUTDOWN THROUGHOUT THE REPORT PERIOD FOR FEEDWATER NOZZLE REPAIR PER NRC BULLETIN NO. 79-13.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

ZION 2

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Illinois/Lake 40 miles N of Chicago
2.	Electric Reliability Council	Main
3.	Type of Reactor	PWR
4.	License Power Level (MWT)	3250
5.	Design Electrical Rating (MWE Net)	1040
6.	Date of Initial Criticality	December 24, 1973
7.	Date of Commercial Operation	September 17, 1974
8.	Condenser Cooling Method	Straight Through
9.	Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10.	Licensee	Commonwealth Edison Co.
11.	Corporate Address	P. O. Box 767 Chicago, Illinois
12.	Corporate Contact	D. L. Peoples, Manager of Nuclear Licensing
13.	Architect/Engineer	Sargent & Lundy
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Various

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Senior Inspector	R. Spessard
19.	IE Resident Inspector	J.E. Kohler
20.	Licensing Project Manager	E. Reeves
21.	Docket Number	50-304
22.	License Number/Date of Issue	DPR-48, 11/14/73
23.	Public Document Room	Zion-Benton Public Library 2600 Emmans Avenue Zion, Illinois 60099

INSPECTION STATUS

ENFORCEMENT STATUS

TECHNICAL SPECIFICATION 6.2.A.11 REQUIRES ADHERENCE TO FIRE PROTECTION PROGRAM IMPLEMENTATION PROCEDURES. CONTRARY TO THE ABOVE, ON OCTOBER 31, 1979, WELDING TOOK PLACE IN THE UNIT 1 TURBINE BUILDING WITHOUT A PORTABLE FIRE EXTINGUISHER AT THE JOB SITE, AS SPECIFIED BY THE APPROVED WELDING AUTHORIZATION CARD.
(7922 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

+ Portable demineralizers continue to be used for processing radioactive liquids due to operational problems with the installed cement solidification system. (79-23)

FACILITY ITEMS(PLANS AND PROCEDURES):

+ Radwaste shipments were resumed to Barnwell, South Carolina on November 28, 1979 following rescission of a shipping ban imposed by the State of South Carolina. (79-23)

MANAGERIAL ITEMS:

+ The licensee has committed to review/revise the procedures and training program for processing, packaging, transfer, and transport of radioactive materials by early 1980. (79-23)

PLANT STATUS

+ The unit was shut down on October 27, 1979, to perform volumetric examination of the feedwater piping per IE Bulletin 79-13. The scheduled startup date is January 24, 1980.

+LAST IE SITE INSPECTION DATE: November 19-20, 1979

+ INSPECTION REPORT NO: 79-23

SUMMARY

INSPECTION SUMMARY

+ Inspection on October 1 through November 5, (79-22): Routine, unannounced resident inspection of licensee action on previous inspection findings, maintenance, plant operations including followup of nonroutine events, review and followup of LER's, surveillance and calibration of safety related components required by TS, radioactive waste systems, feedwater line cracking on Unit 2, fire protection and diesel generator roller pin inspections. The inspection involved 133 inspector hours onsite by one NRC inspector. One item of noncompliance (failure to follow procedures which implement the fire protection program) was identified.

+ Inspection on November 19-20, (79-23): Nonroutine, announced inspection of licensee actions taken in response to IE Bulletin 79-19 "Packaging of Low-Level Radioactive Waste for Transport and Burial" and, as agreed with the State of South Carolina, verification that identified packaging discrepancies were corrected before resumption of radwaste shipments to Barnwell, South Carolina. The inspection involved 28 inspector hours onsite by two NRC inspectors. No items of noncompliance or deviations were identified.

NRC POWER RESTRICTIONS: NONE

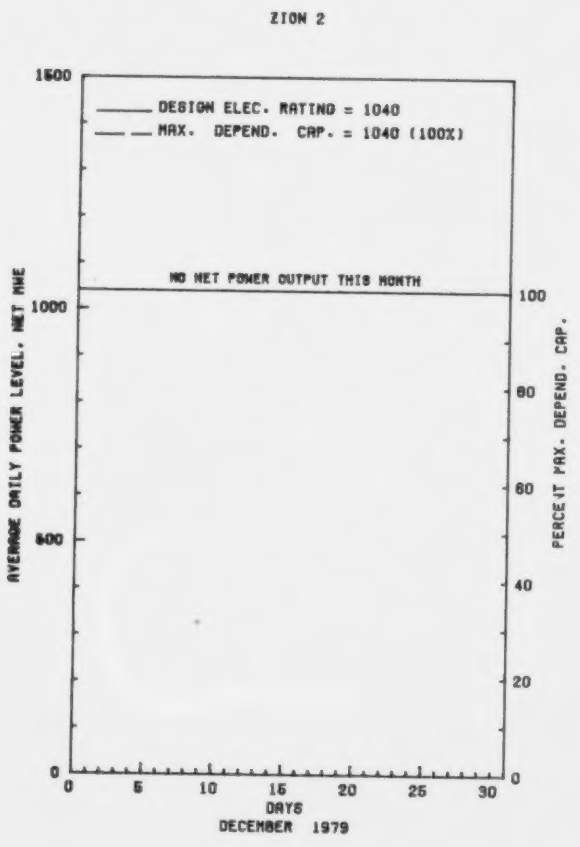
REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
NONE				

REVIEWED BY OMPA LOEB: T. C. Cintula DATE: 2.11.80
REVIEWED BY NRR DOR: Ed A. Reeves DATE: 2-13-80

ZION 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-304
2. Utility Contact: J. Cook (312) 746-2084
3. Licensed Thermal Power (MWt): 3250
4. Nameplate Rating (Gross MWe): 1220 x 0.9 = 1098
5. Design Electrical Rating (Net MWe): 1040
6. Maximum Dependable Capacity (Gross MWe): 1085
7. Maximum Dependable Capacity (Net MWe): 1040
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	8,760.0	46,321.0
12. Hours Reactor Was Critical	.0	6,026.6	33,603.6
13. Reactor Reserve Shutdown Hours	.0	.0	226.1
14. Hours Generator On-Line	.0	5,884.6	32,813.9
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	15,425,148	92,635,318
17. Gross Electrical Energy (MWH)	0	5,036,320	29,721,220
18. Net Electrical Energy (MWH)	0	4,759,996	28,201,453
19. Unit Service Factor	.0	67.2	70.8
20. Unit Availability Factor	.0	67.2	70.8
21. Unit Capacity Factor (MDC Net)	.0	52.2	58.5
22. Unit Capacity Factor (DER Net)	.0	52.2	58.5
23. Unit Forced Outage Rate	100.0	24.6	17.7

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUELING - APRIL 11, 1980 - 7 WEEKS
25. If Shut Down At End of Report Period, Estimated Date of Startup: 01/24/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
16	10/27/79	F	744.0	D	4	79-050	HH	PIPEXX	UNIT DOWN FOR FEEDWATER NOZZLE REPAIR PER NRC BULLETIN #79-13 CONTINUED FROM NOVEMBER.

SUMMARY

RT PERIOD FOR FEEDWATER NOZZLE REPAIR PER NRC BULLETIN NO.

LEGEND

F--Forced
S--Scheduled

A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

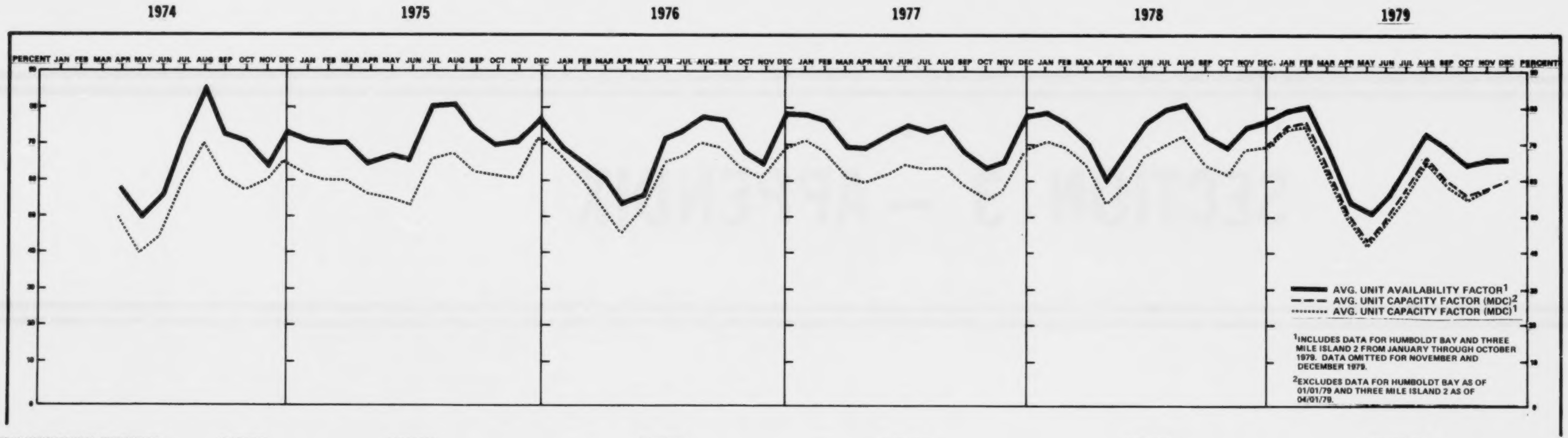
SECTION 3 – APPENDIX

NUCLEAR POWER GENERATION (AVAILABILITY & CAPACITY FACTORS), 1974 TO PRESENT

DEFINITIONS:

AVAILABILITY FACTOR = UNIT AVAILABLE HOURS X 100 / PERIOD HOURS

CAPACITY FACTOR USING MDC NET = NET ELECTRICAL ENERGY GENERATED X 100 / PERIOD HOURS X MAXIMUM DEPENDABLE CAPACITY (NET)



[1] INCLUDES DATA FOR HUMBOLDT BAY AND THREE MILE ISLAND 2 FROM JANUARY THROUGH OCTOBER 1979. DATA OMITTED FOR NOVEMBER AND DECEMBER 1979.

[2] EXCLUDES DATA FOR HUMBOLDT BAY AS OF 01/01/79 AND THREE MILE ISLAND 2 AS OF 04/01/79.

	1974		1975		1976		1977		1978		1979	
NUMBER OF UNITS IN COMMERCIAL OPERATION AND TOTAL CAPACITY IN MWe (NET) AT YEAR END	42 (25,248)		51 (32,378)		55 (37,902)		63 (44,820)		66(47,882)		66 (48,151)	
HIGH/LOW MONTH OF YEAR:	AUGUST/MAY		DECEMBER/JUNE		DECEMBER/APRIL		JANUARY/OCTOBER		AUGUST/APRIL		FEBRUARY/MAY	
STATISTICS OF MONTH: AVAILABILITY FACTOR	86.3	50.3	78.2	66.0	80.1	63.9	79.2	65.1	82.6	61.6	80.7	49.8
CAPACITY FACTOR Using MDC Net	72.8	41.3	73.5	54.1	68.7	46.7	72.4	58.8	73.8	54.2	75.1	42.8

SOURCE: UNVERIFIED DATA

NUCLEAR POWER GENERATION (AVAILABILITY FACTOR, CAPACITY FACTOR & FORCED OUTAGE RATE), 1975 TO PRESENT

DEFINITIONS

AVAILABILITY FACTOR	$\dfrac{\text{UNIT AVAILABLE HOURS X 100}}{\text{PERIOD HOURS}}$
CAPACITY FACTOR USING MDC NET	$\dfrac{\text{NET ELECTRICAL ENERGY GENERATED X 100}}{\text{PERIOD HOURS X MAXIMUM DEPENDABLE CAPACITY (NET)}}$

1975

REPORTING PERIOD	AVG. UNIT AVAIL. FACTOR	AVG. UNIT CAPACITY FACTOR	AVG. FORCED OUTAGE RATE
01/01/75	71.5	59.8	16.2
02/01/75	70.5	61.4	16.6
03/01/75	70.1	63.0	15.0
04/01/75	64.9	56.1	13.7
05/01/75	—	—	15.3
06/01/75	66.2	54.1	19.9
07/01/75	—	—	10.6
08/01/75	80.9	65.3	12.8
09/01/75	74.6	62.3	17.0
10/01/75	—	—	15.3
11/01/75	70.9	60.1	12.1
12/01/75	76.2	—	14.5

1976

REPORTING PERIOD	AVG. UNIT AVAIL. FACTOR	AVG. UNIT CAPACITY FACTOR	AVG. FORCED OUTAGE RATE
01/01/76	70.5	62.8	20.0
02/01/76	66.5	58.0	10.4
03/01/76	58.0	50.5	14.1
04/01/76	53.8	45.8	18.2
05/01/76	57.4	47.7	20.0
06/01/76	73.6	61.2	12.8
07/01/76	75.4	64.4	11.1
08/01/76	80.4	67.8	13.8
09/01/76	78.2	66.3	11.6
10/01/76	69.7	60.5	12.6
11/01/76	64.9	55.3	14.2
12/01/76	90.1	68.7	10.2

1977

REPORTING PERIOD	AVG. UNIT AVAIL. FACTOR	AVG. UNIT CAPACITY FACTOR	AVG. FORCED OUTAGE RATE
01/01/77	79.2	72.4	6.5
02/01/77	75.7	68.6	10.3
03/01/77	71.9	62.5	11.5
04/01/77	70.8	61.5	8.7
05/01/77	73.7	63.2	7.2
06/01/77	77.0	67.0	6.7
07/01/77	75.9	65.5	8.5
08/01/77	76.8	65.6	6.4
09/01/77	69.6	60.1	6.7
10/01/77	65.1	56.8	7.5
11/01/77	66.5	59.0	16.9
12/01/77	79.6	70.9	9.5

1978

REPORTING PERIOD	AVG. UNIT AVAIL. FACTOR	AVG. UNIT CAPACITY FACTOR	AVG. FORCED OUTAGE RATE
01/01/78	80.3	72.9	9.7
02/01/78	77.5	70.5	5.9
03/01/78	72.1	66.3	8.6
04/01/78	61.6	54.2	10.9
05/01/78	69.2	60.5	11.9
06/01/78	77.6	68.3	10.3
07/01/78	81.2	71.8	8.8
08/01/78	82.6	73.8	9.7
09/01/78	72.9	65.1	13.0
10/01/78	69.5	62.8	11.4
11/01/78	75.7	69.3	9.0
12/01/78	77.3	70.0	15.2

1979

REPORTING PERIOD	AVG. UNIT AVAIL. FACTOR	AVG. UNIT CAPACITY FACTOR		AVG. FORCED OUTAGE RATE
01/01/79	80.3	74.4 [1]	75.5 [2]	10.9
02/01/79	80.7	75.1	76.2	5.7
03/01/79	68.4	63.4	64.3	9.2
04/01/79	53.8	51.0	52.5	14.9
05/01/79	49.9	42.8	44.2	19.9
06/01/79	59.2	48.2	49.7	16.5
07/01/79	65.1	56.5	58.2	19.9
08/01/79	74.1	65.4	67.4	16.0
09/01/79	69.1	59.3	61.1	13.9
10/01/79	64.0	55.5	57.2	12.2
11/01/79	66.1	58.3	58.3	15.1
12/01/79	66.6	60.7	60.7	13.0

[1] INCLUDES DATA FOR HUMBOLDT BAY AND THREE MILE ISLAND 2 FROM JANUARY THROUGH OCTOBER 1979. DATA OMITTED FOR NOVEMBER AND DECEMBER 1979.

[2] EXCLUDES DATA FOR HUMBOLDT BAY AS OF 01/01/79 AND THREE MILE ISLAND 2 AS OF 04/01/79.

SOURCE: UNVERIFIED DATA

STATUS OF SPENT FUEL STORAGE CAPABILITY

DATA AS OF 12-31-79

Facility	Core Size [(a)] (No. of Assemblies)	Present Authorized Storage Pool Capacity (Fuel Assemblies)	No. of Assemblies Stored	Remaining Capacity (No. of Assemblies)	Remaining Capacity If Pending Request Approved (No. of Assemblies)	Scheduled Date For Next Refueling	Will Fill Present [(b)] Authorized Capacity
I. Pressurized Water Reactors							
ARKANSAS 1	177	590	176	414		11-80	1988
ARKANSAS 2 (d)	177	486	0	486		02-81	1993
BEAVER VALLEY 1	157	833	0	833		N/S	1995
CALVERT CLIFFS 1	217	1056 (c)	364 (c)	692 (c)		09-80	1984
CALVERT CLIFFS 2	217					11-80	
COOK 1	193	500	193	307	1857	N/S	1985
COOK 2	193					N/S	
CRYSTAL RIVER 3	177	256	60	196	1159	04-80	1982
DAVIS-BESSE 1	177	735	0	735		03-80	1989
FARLEY 1	157	675	0	675		N/S	1990
FORT CALHOUN 1	133	483	157	326		01-80	1985
GINNA	121	595	156	439		03-80	1987
HADDAM NECK	157	1168	340	828		05-80	1994
INDIAN POINT 1	0	288	160	128		N/S	
INDIAN POINT 2	193	482	200	282		N/S	1984
INDIAN POINT 3	193	837	64	773		N/S	1989
KEWAUNEE	121	990	120	870		05-80	1999
MAINE YANKEE	217	953	433	520		01-80	1986
MILLSTONE 2	217	667	144	523		07-80	1986
NORTH ANNA 1	157	400	0	400	966	N/S	1984
NORTH ANNA 2 (d)	157	400	0	400		N/S	
OCONEE 1	177	1224 (l)	545	679 (l)	679	N/S	1984
OCONEE 2	177					02-80	1984
OCONEE 3	177					N/S	1984
PALISADES	204	798	273	525		N/S	1985
POINT BEACH 1	121	1502 (c)	212 (c)	1290 (c)		N/S	1995
POINT BEACH 2	121					03-80	1995
PRAIRIE ISLAND 1	121	687 (c)	280 (c)	407 (c)		06-80	1984
PRAIRIE ISLAND 2	121					01-80	1984
RANCHO SECO 1	177	579	112	467		01-80	1987
ROBINSON 2	157	272	120	152 (e)		05-80	1980 (g)
SALEM 1	193	264	40	224	1170	09-80	1982
SAN ONOFRE 1	157	216	58	158		04-80	1983
ST LUCIE 1	217	728	60	668		03-80	1990
SURRY 1	157	1044 (c)	436 (c)	608 (c)		06-80	1983
SURRY 2	157					N/S	
THREE MILE ISLAND 1	177	752	212	540		N/S	1985
THREE MILE ISLAND 2	177	442	0	442		N/S	1986
TROJAN	193	651	64	587		04-80	1988
TURKEY POINT 3	157	621	346 (c)	275 (c)		N/S	1987
TURKEY POINT 4	157					N/S	1987
YANKEE-ROWE 1	76	391	149	242	572	N/S	1985
ZION 1	193	868 (c)	436 (c)	432 (c)	1676	N/S	1983
ZION 2	193					04-80	1983

Facility	Core Size [(a)] (No. of Assemblies)	Present Authorized Storage Pool Capacity (Fuel Assemblies)	No. of Assemblies Stored	Remaining Capacity (No. of Assemblies)	Remaining Capacity If Pending Request Approved (No. of Assemblies)	Scheduled Date For Next Refueling	Will Fill Present [(b)] Authorized Capacity
II. Boiling Water Reactors							
BIG ROCK POINT 1	84	193	88	105	353	10-80	1983
BROWNS FERRY 1	764	3471	576	2895		01-80	1995
BROWNS FERRY 2	764	1080	400	680		09-80	1995
BROWNS FERRY 3	764	1528	354	1174		N/S	1995
BRUNSWICK 1	560	(f)	154(PWR)+320(BWR)			04-80	1986
BRUNSWICK 2	560	(f)	91(PWR)+132(BWR)			02-80	1986
COOPER STATION	548	2366	284	2082		03-80	1995
DRESDEN 1	464	672	221	451		N/S	1980
DRESDEN 2	724	2840 (c)	1069 (c)	1771 (c)	6491 (c)	N/S	1985
DRESDEN 3	724					03-80	1984
DUANE ARNOLD	368	2050	276	1774		02-80	1998
FITZPATRICK	560	760	268	492	1752	04-80	1981
HATCH 1	560	840	260	580		N/S	1988
HATCH 2	560	1120	0	1120		N/S	
HUMBOLDT BAY	172	487	251	236		N/S	
LA CROSSE	72	134	113	21	327	02-80	1983
MILLSTONE 1	580	2184	776	1408		09-80	1986
MONTICELLO	484	2237	728	1509		02-80	1990
NINE MILE POINT 1	532	1984	660	1324	2349	N/S	1990
OYSTER CREEK 1	560	1800	620	1180		01-80	1987
PEACH BOTTOM 2	764	2816	618	2198		03-80	1990
PEACH BOTTOM 3	764	2816	712	2104		03-81	1991
PILGRIM 1	580	2320	580	1740		01-80	1990
QUAD CITIES 1	724	1460	596	864		09-80	1985
QUAD CITIES 2	724	1460	492	968		N/S	1985
VERMONT YANKEE 1	368	2000	894	1106		N/S	1988
III. Independent Spent Fuel Storage Installations (h)							
Morris Operations		750 MTU (j)	310	390 MTU (j)	1490 MTU (j)		
NFS(i)		250 MTU	170 MTU	80 MTU			

(a) At each refueling outage approximately 1/3 of a PWR core and 1/4 of a BWR core is off-loaded.
(b) Some of these dates have been adjusted by staff assumptions.
(c) This is the total for both units.
(d) Plant not in commercial operation.
(e) Some spent fuel stored at Brunswick.
(f) Authorized a total 2088 BWR and 304 PWR assemblies for both pools.
(g) Robinson 2 assemblies being shipped to Brunswick for storage.
(h) Capacity is in metric tons of uranium; 1 MTU = 2 PWR assemblies or 5 BWR assemblies.
(i) No longer accepting spent fuel.
(j) Racked for 700 MTU.
(k) Reserved
(l) This is the station total.

N/S = Not Scheduled Yet

The total of Reactor years of experience is as the sum of all calendar days for each unit, from the date that electricity was first generated until a final shutdown date or the status date, whichever comes first, divided by 365.25 days/year.

If a date is unknown, the first day of the first month of operation is substituted. Units which have not yet generated electricity but which are licensed are listed but not included in the computation.

REACTOR-YEARS OF EXPERIENCE

(INFORMATION INCLUDES BOTH LICENSED AND NON–LICENSED ELECTRICAL UNITS).

LICENSED OPERATING ELECTRICAL PRODUCING UNITS

UNIT NAME	FIRST ELEC. GENERATION	YEARS EXPERIENCE
ARKANSAS 1	08/01/74	5.42
ARKANSAS 2	12/26/78	1.02
BEAVER VALLEY 1	06/14/76	3.55
BIG ROCK POINT 1	12/08/62	17.07
BROWNS FERRY 1	10/15/73	6.21
BROWNS FERRY 2	08/28/74	5.34
BROWNS FERRY 3	09/12/76	3.30
BRUNSWICK 1	12/04/76	3.07
BRUNSWICK 2	04/29/75	4.68
CALVERT CLIFFS 1	01/03/75	4.99
CALVERT CLIFFS 2	12/07/76	3.07
COOK 1	02/10/75	4.89
COOK 2	03/22/78	1.78
COOPER STATION	05/10/74	5.65
CRYSTAL RIVER 3	01/30/77	2.92
DAVIS-BESSE 1	08/28/77	2.34
DRESDEN 1	04/15/60	19.71
DRESDEN 2	04/13/70	9.72
DRESDEN 3	07/22/71	8.45
DUANE ARNOLD	05/19/74	5.62
FARLEY 1	08/18/77	2.37
FITZPATRICK	02/01/75	4.91
FORT CALHOUN 1	08/25/73	6.35
FORT ST VRAIN	12/11/76	3.06
GINNA	12/02/69	10.08
HADDAM NECK	08/07/67	12.40
HATCH 1	11/11/74	5.14
HATCH 2	09/22/78	1.28
INDIAN POINT 2	06/26/73	6.52
INDIAN POINT 3	04/27/76	3.68
KEWAUNEE	04/08/74	5.73
LA CROSSE	04/26/68	11.68
MAINE YANKEE	11/08/72	7.15
MILLSTONE 1	11/29/70	9.09
MILLSTONE 2	11/09/75	4.15
MONTICELLO	03/05/71	8.83
NINE MILE POINT 1	11/09/69	10.14
NORTH ANNA 1	04/17/78	1.71
OCONEE 1	05/06/73	6.66
OCONEE 2	12/05/73	6.07
OCONEE 3	09/01/74	5.33
OYSTER CREEK 1	09/23/69	10.27
PALISADES	12/31/71	8.00
PEACH BOTTOM 2	02/18/74	5.87
PEACH BOTTOM 3	09/01/74	5.33
PILGRIM 1	07/19/72	7.45
POINT BEACH 1	11/06/70	9.15
POINT BEACH 2	08/02/72	7.41
PRAIRIE ISLAND 1	12/04/73	6.08
PRAIRIE ISLAND 2	12/21/74	5.03
QUAD CITIES 1	04/12/72	7.72
QUAD CITIES 2	05/23/72	7.61
RANCHO SECO 1	10/13/74	5.22
ROBINSON 2	09/26/70	9.26
SALEM 1	12/25/76	3.02

LICENSED OPERATING ELECTRICAL PRODUCING UNITS - CONT.

UNIT NAME	FIRST ELEC. GENERATION	YEARS EXPERIENCE
SAN ONOFRE 1	07/16/67	12.46
ST LUCIE 1	05/07/76	3.65
SURRY 1	07/04/72	7.49
SURRY 2	03/10/73	6.81
THREE MILE ISLAND 1	06/19/74	5.54
TROJAN	12/23/75	4.02
TURKEY POINT 3	11/02/72	7.16
TURKEY POINT 4	06/21/73	6.53
VERMONT YANKEE 1	09/20/72	7.28
YANKEE-ROWE 1	11/10/60	19.14
ZION 1	06/28/73	6.51
ZION 2	12/26/73	6.02
TOTAL		441.16 YEARS

PERMANENTLY OR INDEFINITELY SHUTDOWN UNITS

UNIT NAME	FIRST ELEC. GENERATION	SHUTDOWN OR ELECTRICAL CESSATION DATE	YEARS EXPERIENCE
BONUS	08/14/64	06/01/68	3.80
CVTR	12/18/63	01/01/67	3.04
ELK RIVER	08/24/63	02/01/68	4.44
FERMI 1	08/05/66	11/29/72	6.32
HALLAM	05/29/63	09/01/64	1.26
HUMBOLDT BAY*	04/18/63	07/02/76	13.21
INDIAN POINT 1	09/16/62	10/31/74	12.12
PATHFINDER	07/25/66	10/01/67	1.19
PEACH BOTTOM 1	01/27/67	11/01/74	7.76
PIQUA	11/04/63	01/01/66	2.16
THREE MILE ISLAND 2*	04/21/78	03/28/79	.93
TOTAL			56.23 YEARS

*NOTE: HUMBOLDT BAY AND THREE MILE ISLAND 2 HAVE BEEN CHANGED FROM THE CATEGORY OF "LICENSED OPERATING ELECTRICAL PRODUCING UNITS" TO THE CLASSIFICATION OF A "PERMANENTLY OR INDEFINITELY SHUTDOWN UNITS". FOR THE COMPUTATION OF REACTOR YEARS OF EXPERIENCE, THEIR EFFECTIVE SHUTDOWN OR ELECTRICAL CESSATION DATE IS AS LISTED. BECAUSE, IN PREVIOUS EDITIONS, THESE TWO UNITS HAD BEEN CONSIDERED TO BE OPERATING UNITS, THE TOTAL YEARS OF OPERATING EXPERIENCE WILL NOT BE CONSISTENT WITH THE VALUE PUBLISHED FOR NOVEMBER 1, 1979.

NON–POWER REACTORS IN THE U.S.

1. RESEARCH REACTORS

State	Location	Licensee	Reactor Type	Docket	License Number	Date Issued	Authorized Power Level (KW)
Alabama	1. Tuskegee	Tuskegee Institute	AGN 201 No. 102	50-406	R-122	08-30-74	.0001
Arizona	2. Tucson	U. of Arizona	TRIGA Mk. I	50-113	R-52	12-05-58	100
California	3. Los Angeles	U. of California	Argonaut	50-142	R-71	10-03-60	100
	4. Berkeley	U. of California	TRIGA Mark III	50-224	R-101	08-10-66	1000
	5. Irvine	U. of California	TRIGA Mark I	50-326	R-116	11-24-69	250
	6. Santa Barbara	U. of California	L-77	50-433	R-124	12-03-74	.01
	7. Canoga Park	Rockwell International Corporation	L-77	50-94	R-40	05-17-58	.01
	8. Hawthorne	Northrop Corporate Laboratories	TRIGA Mark F	50-187	R-90	03-04-63	1000
	9. San Luis Obispo	California State Polytechnic College	AGN 201 No. 100	50-394	R-121	05-16-73	.0001
	10. Alameda County	General Electric Co.	NTR	50-73	R-33	10-31-57	100
	11. San Diego	General Atomic Co.	TRIGA Mark I	50-89	R-38	05-03-58	250
	12. San Diego	General Atomic Co.	TRIGA Mark F	50-163	R-67	07-01-60	1500
	13. San Ramon	Aerotest Operations, Inc.	TRIGA (Industrial)	50-228	R-98	07-02-65	250
Colorado	14. Jefferson County	U.S. Geological Survey. Dept. of Interior	TRIGA Mark I	50-274	R-113	02-24-69	1000
Delaware	15. Newark	U. of Delaware	AGN-201 No. 113	50-98	R-43	07-03-58	.0001
District of Columbia	16. –	Catholic U.	AGN-201 No. 101	50-77	R-31	11-15-67	.0001
Florida	17. Gainesville	U. of Florida	Argonaut	50-83	R-56	05-21-59	100
Georgia	18. Atlanta	Georgia Institute of Technology	AGN-201 No. 104	50-276	R-111	04-19-68	.0001
	19. Atlanta	Georgia Institute of Technology	Heavy Water	50-160	R-97	12-29-64	5000
Idaho	20. Pocatello	Idaho State U.	AGN-201 No. 103	50-284	R-110	10-11-67	.0001
Illinois	21. Urbana	U. of Illinois	Advanced TRIGA	50-151	R-115	07-22-69	1500
	22. Urbana	U. of Illinois	LOPRA	50-356	R-117	12-27-71	10
	23. Zion	Westinghouse Electric Corporation	NTR	50-87	R-119	01-28-72	10
Indiana	24. West Lafayette	Purdue U.	Lockheed	50-182	R-87	08-16-62	10

RESEARCH REACTORS – CONT.

State	Location	Licensee	Reactor Type	Docket	License Number	Date Issued	Authorized Power Level (KW)
Iowa	25. Ames	Iowa State U.	UTR-10	50-116	R-59	10-16-59	10
Kansas	26. Lawrence	U. of Kansas	Lockheed	50-148	R-78	06-23-61	250
	27. Manhattan	Kansas State U.	TRIGA	50-188	R-88	10-16-62	250
Maryland	28. College Park	U. of Maryland	TRIGA	50-166	R-70	10-14-60	250
	29. Bethesda	Armed Forces Radiobiology Research Institute	TRIGA	50-170	R-84	06-26-62	1000
Massachusetts	30. Lowell	U. of Lowell	GE	50-223	R-125	12-24-74	1000
	31. Cambridge	Massachusetts Institute of Technology	Heavy Water Reflected	50-20	R-37	06-09-58	5000
	32. Worcester	Worcester Polytechnic Institute	GE	50-134	R-61	12-16-59	10
Michigan	33. Ann Arbor	U. of Michigan	Pool	50-2	R-28	09-13-57	2000
	34. Lansing	Michigan State U.	TRIGA Mark I	50-294	R-114	03-21-69	250
	35. Midland	Dow Chemical Co.	TRIGA Mark I	50-264	R-108	07-03-67	100
Missouri	36. Rolla	U. of Missouri	Pool	50-123	R-79	11-21-61	200
	37. Columbia	U. of Missouri	Tank	50-186	R-103	10-11-66	10,000
Nebraska	38. Omaha	V.A. Hospital	TRIGA	50-131	R-57	06-26-59	18
New Mexico	39. Albuquerque	U. of New Mexico	AGN-201M No. 112	50-252	R-102	09-17-66	.005
New York	40. Buffalo	State U. of New York	PULSTAR	50-57	R-77	03-24-61	2000
	41. Sterling Forest	Union Carbide Corp.	Pool	50-54	R-81	09-07-61	5000
	42. New York City	Columbia U.	TRIGA Mark II	50-208	R-128	04-14-77	250
	43. Ithaca	Cornell U.	ZPR	50-97	R-89	12-11-62	.1
	44. Ithaca	Cornell U.	TRIGA Mk. II	50-157	R-80	01-11-62	100
	45. Riverdale	Manhattan College	Tank	50-199	R-94	03-24-64	.0001
North Carolina	46. Raleigh	North Carolina State U.	PULSTAR	50-297	R-120	08-25-72	1000
Ohio	47. Columbus	Ohio State U.	Pool	50-150	R-75	02-24-61	10
Oklahoma	48. Norman	U. of Oklahoma	AGN-211 No. 102	50-112	R-53	12-29-58	.015
Oregon	49. Corvallis	Oregon State U.	AGN-201 No. 114	50-106	R-51	11-13-58	.0001
	50. Corvallis	Oregon State U.	TRIGA Mark II	50-243	R-106	03-07-67	1000
	51. Portland	Reed Institute	TRIGA Mk. I	50-288	R-112	07-02-68	250
Pennsylvania	52. University Park	Pennsylvania State U.	TRIGA Mk. III	50-5	R-2	07-08-55	1000

NON–POWER REACTORS IN THE U.S.

RESEARCH REACTORS – CONT.

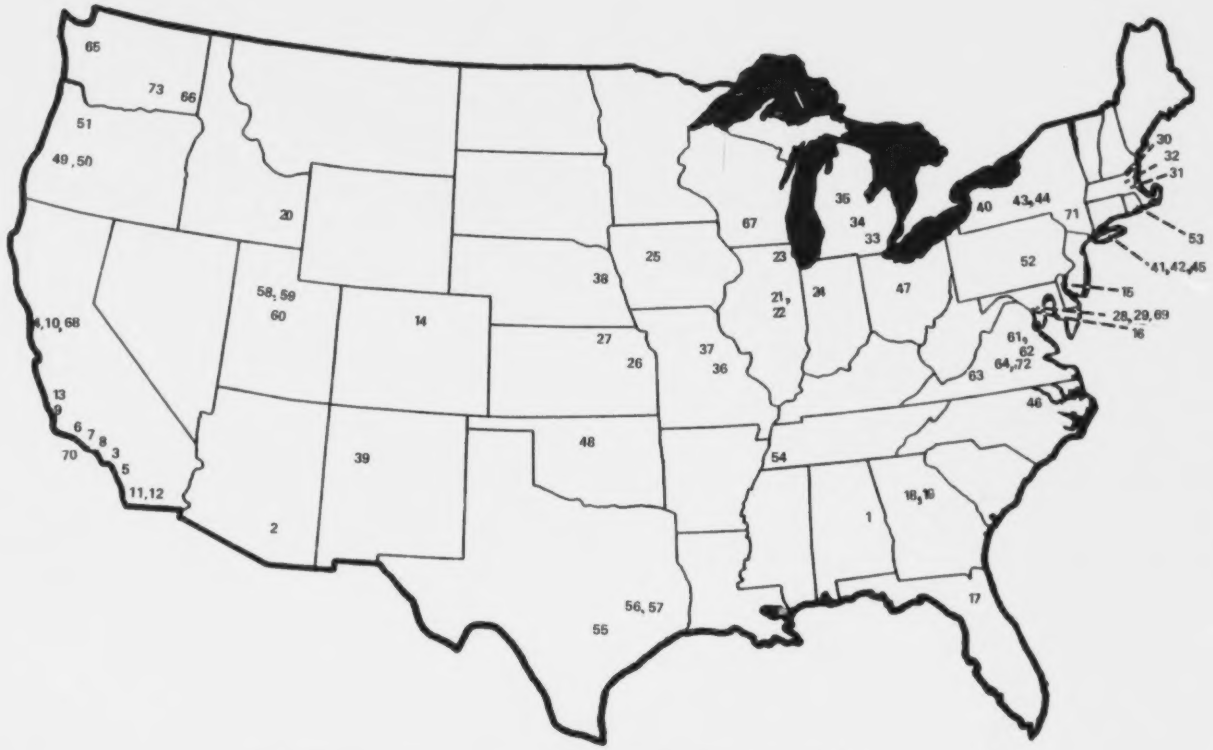
State	Location	Licensee	Reactor Type	Docket	License Number	Date Issued	Authorized Power Level (KW)
Rhode Island	53. Ft. Kearney, Narragansett	Rhode Island & Providence Plantations AEC	GE Pool	50-193	R-95	07-21-64	2000
Tennessee	54. Shelby County	Memphis State U.	AGN-201 No. 108	50-538	R-127	12-10-76	.0001
Texas	55. Austin	U. of Texas	TRIGA Mk. I	50-192	R-92	08-02-63	250
	56. College Station	Texas A&M U.	TRIGA	50-128	R-83	12-07-61	1000
	57. College Station	Texas A&M U.	AGN- 201M No. 106	50-59	R-23	08-26-57	.005
Utah	58. Salt Lake City	U. of Utah	AGN-201M No. 107	50-72	R-25	09-12-57	.005
	59. Salt Lake City	U. of Utah	TRIGA Mark I	50-407	R-126	09-30-75	100
	60. Provo	Brigham Young U.	L-77	50-262	R-109	09-07-67	.01
Virgina	61. Charlottesville	U. of Virgina	Pool	50-62	R-66	06-27-60	2000
	62. Charlottesville	U. of Virginia	Cavalier	50-396	R-123	09-24-74	.1
	63. Blacksburg	Virginia Polytechnic Institute	UTR-10	50-124	R-62	12-18-59	100
	64. Lynchburg	Babcock & Wilcox Co.	LPR	50-99	R-47	09-05-58	1000
Washington	65. Seattle	U. of Washington	Argonaut	50-139	R-73	03-31-61	100
	66. Pullman	Washington State U.	TRIGA	50-27	R-76	03-06-61	1000
Wisconsin	67. Madison	U. of Wisconsin	TRIGA	50-156	R-74	11-23-60	1000

2. EXPERIMENTAL AND TEST REACTORS

State	Location	Licensee	Reactor Type	Docket	License Number	Date Issued	Authorized Power Level (MWt)
California	68. Alameda Co.	General Electric Co.	GETR	50-70	TR-1	01-07-59	50
Maryland	69. Gaithersburg	National Bureau of Standards	TEST	50-184	TR-5	11-13-67 POL 06-30-70 OL	10

3. CRITICAL EXPERIMENT FACILITIES

State	Location	Licensee	Docket	License Number	Date Issued	Experiments
California	70. Ventura Co.	Rockwell International Corp. (Possesion Only)	50-147	CX-17	10-03-60	AETR Experiments (Terminal Requested).
New York	71. Schenectady	Rensselaer Polytechnic Institute	50-225	CX-22	07-03-64	
Virginia	72. Lynchburg	Babcock & Wilcox Co.	50-13	CX-10	10-22-58	NMSR Experiments
Washington	73. Richland	Battelle Memorial Institute	50-360	CX-26	11-29-71	Plutonium Recycle Critical Facility



Source: NRC Facilities License Application Record (FLAR) 06-30-77.

ABBREVIATIONS

AGN	Aerojet General Nuclear	These reactors were constructed by Aeroject at San Ramon, California, and then transferred to the current location.
ZPR	Zero Power Reactor	
LPR	Low Power Reactor	

DISTRIBUTION LIST–NRC

INTERNAL DISTRIBUTION

Office of the Commissioners	5
Atomic Safety and Licensing Appeal Panel	3
Advisory Committee on Reactor Safeguards	6
Office of Inspector and Auditor	1
Office of Policy Evaluation	1
Office of the General Counsel	1
Office of Public Affairs	21
Office of Congressional Affairs	3
Office of the Executive Director for Operations	7
Office of Administration	2
Office of the Executive Legal Director	2
Office of the Controller	2
Office for Analysis and Evaluation of Operational Data	2
Office of International Programs	3
Office of State Programs	1
Office of Management and Program Analysis	14
Office of Standards Development	1
Office of Nuclear Material Safety and Safeguards	2
– Division of Fuel Cycle and Material Safety	3
– Division of Safeguards	2
Office of Nuclear Reactor Regulation	5
– Division of Project Management	6
– Division of Operating Reactors	105
– Division of Systems Safety	4
– Division of Site Safety and Environmental Analysis	18
Office of Nuclear Regulatory Research	4
Office of Inspection and Enforcement	7
– Region I	11
– Region II	10
– Region III	10
– Region IV	10
– Region V	7
	279

EXTERNAL DISTRIBUTION

SPECIAL REQUESTS

Air Force	1
Congress	10
Department of Energy	32
Government Accounting Office	1
Environmental Protection Agency	1
Electric Power Research Institute	1
Argonne National Laboratory	3
Federal Power Commission	1
Bureau of Mines	1
Department of Agriculture	1
Department of Commerce	1
Department of the Interior	1
	54

OTHER

GPO Depository	440
GPO Stores	75
National Technical Information Service	25
Subscriptions (NTIS)	250
Colleges and Libraries (including Public Document Rooms)	128
Utilities and Other Requests	227
	1145

15/4-2

Nuclear Regulatory Commission

NUREG 0020
VOL. 4 NO. 2
FEBRUARY 1980

OPERATING UNITS STATUS REPORT

DATA AS OF 01-31-80

NEWARK PUBLIC LIBRARY
APR 1 0 1980
U. S. GOVT. PUBS.
DEPOSITORY 376

LICENSED OPERATING REACTORS

Y3.N88:

Available from
GPO Sales Program
Division of Technical Information and Document Control
U. S. Nuclear Regulatory Commission
Washington, D.C. 20555
and
National Technical Information Service
Springfield, Virginia 22161
703-326-8543
A Year's Subscription, 12 Issues, for this Publication is
$120.00 North American Continent ($240 00 Other).
Multiple Subscriptions Entered at the Same Time to the Same Address
Cost $90.00 North American Continent ($180.00 Other) for each Additional Subscription.

Single copies are $12.50 North American Continent ($25.00 Other).

This document is provided free of charge to those who participate in the data collection program, and to the Nuclear Regulatory Commission's local public document rooms.

Issued by the U.S. Nuclear Regulatory Commission

PUBLISHED: 03/26/80

Nuclear Regulatory Commission

NUREG 0020
VOL. 4 NO. 2
FEBRUARY 1980

OPERATING UNITS STATUS REPORT

DATA AS OF 01-31-80

LICENSED OPERATING REACTORS

AUTHORIZATION AND CLEARANCE

THE U.S. NUCLEAR REGULATORY COMMISSION'S OFFICE OF MANAGEMENT AND PROGRAM ANALYSIS PUBLISHES THIS MONTHLY STATUS REPORT "AS PART OF THE REPORTING REQUIREMENTS IN SECTION 50.36 OF 10 CFR PART 50 UNDER GAO CLEARANCE NUMBER B-180225, WITH AN EXPIRATION DATE OF SEPTEMBER 30, 1981," AS STATED IN THE OCTOBER 3, 1978 LETTER FROM JOHN M. LOVELADY, ASSISTANT DIRECTOR, GENERAL GOVERNMENT DIVISION, U. S. GENERAL ACCOUNTING OFFICE, TO J.M. FELTON, DIRECTOR, DIVISION OF RULES AND RECORDS, U.S. NUCLEAR REGULATORY COMMISSION.

STATEMENT OF PURPOSE

The U. S. Nuclear Regulatory Commission's monthly OPERATING UNITS STATUS REPORT – LICENSED OPERATING REACTORS, provides data on the operation of nuclear units as timely and accurately as possible. This information is collected by the Office of Management and Program Analysis from the Headquarters Staff of NRC's Office of Inspection and Enforcement, from NRC's Regional Offices, and from utilities. Since all of the data concerning operation of the units is provided by the utility operators less than two weeks after the end of the month, necessary corrections to published information are shown on the ERRATA page.

This report is divided into three sections: the first contains monthly highlights and statistics for commercial operating units, and errata from previously reported data; the second is a compilation of detailed information on each unit, provided by NRC Regional Offices and IE Headquarters; and the third section is an appendix containing nuclear availability and capacity factors, forced outage rate statistics, spent fuel storage capability, reactor-years of experience and non-power reactors in the United States.

Throughout the report, statistical factors for periods greater than a report month, or for more than one unit, are computed as the arithmetic average of each unit's individual operating statistics. The statistical factors for each unit for the report month are computed from actual power production for the month. The factors for the life-span of each unit (the "Cumulative" column) are reported by the utility and are not entirely re-computed by NRC. Utility power production data is checked for consistency with previously submitted statistics.

It is hoped this status report proves informative and helpful to all agencies and individuals interested in analyzing trends in the nuclear industry which might have safety implications, or in maintaining an awareness of the U. S. energy situation as a whole.

PHILIP A. ROSS
REVIEW COORDINATOR

DEBBIE NICHOLS
TECHNICAL ASSISTANT

TABLE OF CONTENTS

GLOSSARY ii
INDEX TO LICENSED UNITS iii

SECTION 1 – CURRENT DATA SUMMARIES

MONTHLY HIGHLIGHTS OF COMMERCIAL NUCLEAR POWER UNITS 1-2

- Licensed Power Reactors 1-2
- Power Generation 1-2
- Actual vs. Potential Energy Production 1-2
- Outage Data 1-2
- Derated Units 1-2
- Shutdowns - Number and Reasons 1-3
- Unit Availability, Capacity, and Forced Outage Rate 1-3
- Refueling Schedule 1-3

AVERAGE CAPACITY FACTORS BY VENDOR 1-4

AVERAGE DAILY POWER LEVEL FOR ALL COMMERCIAL OPERATING UNITS 1-5

ERRATA 1-7

SECTION 2 – OPERATING POWER REACTORS

ARKANSAS 1 THROUGH ZION 2 PAGES A–2 TO Z–5

- Facility Data
- Inspection and Enforcement Status
- Reports Received from Licensee
- Operating Status
- Average Daily Power Level (MWe)
- Unit Shutdowns/Reductions

SECTION 3 – APPENDIX

STATUS OF SPENT FUEL STORAGE CAPABILITY 3-2
REACTOR-YEARS OF EXPERIENCE 3-3
NON-POWER REACTORS IN THE U.S. 3-4

GLOSSARY

AVERAGE DAILY POWER LEVEL (MWe)	The net electrical energy generated during the day (measured from 0001 to 2400 hours inclusive) in megawatts hours divided by 24 hours.
LICENSED THERMAL POWER (MWt)	The maximum thermal power of the reactor authorized by the NRC, expressed in megawatts.
DATE OF COMMERCIAL OPERATION	Date unit was declared by utility owner to be available for the regular production of electricity; usually related to satisfactory completion of qualification tests as specified in the purchase contract and to accounting policies and practices of utility.
DESIGN ELECTRICAL RATING (DER) (Net MWe)	The nominal net electrical output of the unit specified by the utility and used for the purpose of plant design.
FORCED OUTAGE	An outage required to be initiated no later than the weekend following discovery of an off-normal condition.
FORCED OUTAGE HOURS	The clock hours during the report period that a unit is unavailable due to forced outages.
GROSS ELECTRICAL ENERGY GENERATED (MWH)	Electrical output of the unit during the report period as measured at the output terminals of the turbine generator, in megawatts hours.
GROSS HOURS	The clock hours from the beginning of a specified situation until its end. For outage durations, the clock hours during which the unit is not in power production.
GROSS THERMAL ENERGY GENERATED (MWH)	The thermal energy produced by the unit during the report period as measured or computed by the licensee in megawatt hours.
HOURS GENERATOR ON-LINE	Also, "Unit Service Hours." The total clock hours in the report period during which the unit operated with breakers closed to the station bus. These hours added to the total outage hours experienced by the unit during the report period, shall equal the hours in the report period.
HOURS IN REPORTING PERIOD	For units in power ascension at the end of the period, the gross hours from the beginning of the period or the first electrical production, whichever comes last, to the end of the period. For units in commercial operation at the end of the period, the gross hours from the beginning of the period or of commercial operation, whichever comes last, to the end of the period or decommissioning, whichever comes first.
HOURS REACTOR CRITICAL	The total clock hours in the report period during which the reactor sustained a controlled chain reaction.
MAXIMUM DEPENDABLE CAPACITY (GROSS) (MDC Gross) (Gross MWe)	Dependable main-unit gross capacity, winter or summer, whichever is smaller. The dependable capacity varies because the unit efficiency varies during the year due to cooling water temperature variations. It is the gross electrical output as measured at the output terminals of the turbine generator during the most restrictive seasonal conditions (usually summer).
MAXIMUM DEPENDABLE CAPACITY (NET) (MDC Net) (Net MWe)	Maximum Dependable Capacity (Gross) less the normal station service loads.
NAMEPLATE RATING (Gross MWe)	The nameplate power designation of the generator in megavolt amperes (MVA) times the nameplate power factor of the generator. NOTE: The nameplate rating of the generator may not be indicative of the maximum or dependable capacity, since some other item of equipment of a lesser rating (e.g., turbine) may limit unit output.
NET ELECTRICAL ENERGY GENERATED	Gross electrical output of the unit measured at the output terminals of the turbine generator during the reporting period, minus the normal station service electrical energy utilization. If this quantity is less than zero, a negative number should be recorded.
OUTAGE	A situation in which no electrical production takes place.
OUTAGE DATE	As reported on Appendix D of Reg. Guide 1.16, the date of the start of the outage. If continued from a previous month, report the same outage date but change "Method of Shutting Down Reactor" to "4 (Other)" and add a note: "Continued from previous month."
OUTAGE DURATION	The total clock hours of the outage measured from the beginning of the report period or the outage, whichever comes last, to the end of the report period or the outage, whichever comes first.
OUTAGE NUMBER	A number unique to the outage assigned by the licensee. The same number is reported each month in which the outage is in progress. One format is "76-05" for the fifth outage to occur in 1976.
PERIOD HOURS	See "Hours in Reporting Period."
POWER REDUCTION	A reduction in the Average Daily Power Level of more than 20% from the previous day. All power reductions are defined as outage of zero hours durations for the purpose of computing unit service and availability factors, and forced outage rate.
REACTOR AVAILABLE HOURS	The total clock hours in the report period during which the reactor was critical or was capable of being made critical. (Reactor Reserve Shutdown Hours + Hours Reactor Critical.)
REACTOR AVAILABILITY FACTOR	$\frac{\text{Reactor Available Hours x 100}}{\text{Period Hours}}$
REACTOR RESERVE SHUTDOWN	The cessation of criticality in the reactor for administrative or other similar reasons when operation could have been continued.
REACTOR RESERVE SHUTDOWN HOURS	The total clock hours in the report period during which the reactor is in reserve shutdown mode. NOTE: No credit is given for NRC imposed shutdowns.
REACTOR SERVICE FACTOR	$\frac{\text{Hours Reactor Critical x 100}}{\text{Period Hours}}$
REPORT PERIOD	Usually, the preceding calendar month. Can also be the preceding calendar year, (Year-to-Date), or the life-span of a unit (cumulative).
RESTRICTED POWER LEVEL	Maximum net electrical generation to which the unit is restricted during the report period due to the state of equipment, external conditions, administrative reasons, or a direction by NRC.
SCHEDULED OUTAGE	Planned removal of a unit from service for refueling, inspection, training, or maintenance. Those outages which do not fit the definition of "Forced Outage" perforce are "Scheduled Outages."
STARTUP AND POWER ASCENSION TEST PHASE	Period following initial criticality during which the unit is tested at successively higher levels, culminating with operation at full power for a sustained period and completion of warranty runs. Following this phase, the utility generally considers the unit to be available for commercial operation.
UNIT	The set of equipment uniquely associated with the reactor, including turbine generators, and ancillary equipment, considered as a single electrical energy production facility.
UNIT AVAILABLE HOURS	The total clock hours in the report period during which the unit operated on-line or was capable of such operation. (Unit Reserve Shutdown Hours + Hours Generator On-Line.)
UNIT AVAILABILITY FACTOR*	$\frac{\text{Unit Available Hours x 100}}{\text{Period Hours}}$
UNIT CAPACITY FACTORS	
– Using Licensed Thermal Power	$\frac{\text{Gross Thermal Energy Generated x 100}}{\text{Period Hours x Lic. Thermal Power}}$
– Using Nameplate Rating	$\frac{\text{Gross Electrical Energy Generated x 100}}{\text{Period Hours x Nameplate Rating}}$
– Using DER	$\frac{\text{Net Electrical Energy Generated x 100}}{\text{Period Hours x DER}}$
– Using MDC Gross	$\frac{\text{Gross Electrical Energy Generated x 100}}{\text{Period Hours x MDC Gross}}$
– Using MDC Net	$\frac{\text{Net Electrical Energy Generated x 100}}{\text{Period Hours x MDC Net}}$

NOTE: if MDC Gross and/or MDC Net have not been determined, the DER is substituted for this quantity for Unit Capacity Factor calculations.

UNIT FORCED OUTAGE RATE	$\frac{\text{Forced Outage Hours}}{\text{Unit Service Hours + Forced Outage Hours}}$
UNIT RESERVE SHUTDOWN	The removal of the unit from on-line operation for economic or other similar reasons when operation could have been continued.
UNIT RESERVE SHUTDOWN HOURS	The total clock hours in the report period during which the unit was in reserve shutdown mode.
UNIT SERVICE FACTOR	$\frac{\text{Unit Service Hours x 100}}{\text{Period Hours}}$
UNIT SERVICE HOURS	See "Hours Generator On-Line."

INDEX TO LICENSED UNITS

ARKANSAS 1	A-3
ARKANSAS 2	A-5
BEAVER VALLEY 1	B-3
BIG ROCK POINT 1	B-5
BROWNS FERRY 1	B-7
BROWNS FERRY 2	B-9
BROWNS FERRY 3	B-11
BRUNSWICK 1	B-13
BRUNSWICK 2	B-15
CALVERT CLIFFS 1	C-3
CALVERT CLIFFS 2	C-5
COOK 1	C-7
COOK 2	C-9
COOPER STATION	C-11
CRYSTAL RIVER 3	C-13
DAVIS-BESSE 1	D-3
DRESDEN 1	D-5
DRESDEN 2	D-7
DRESDEN 3	D-9
DUANE ARNOLD	D-11
FARLEY 1	F-3
FITZPATRICK	F-5
FORT CALHOUN 1	F-7
FORT ST VRAIN	F-9
GINNA	G-3
HADDAM NECK	H-3
HATCH 1	H-5
HATCH 2	H-7
HUMBOLDT BAY	H-9
INDIAN POINT 1	I-3
INDIAN POINT 2	I-5
INDIAN POINT 3	I-7
KEWAUNEE	K-3
LA CROSSE	L-3
MAINE YANKEE	M-3
MILLSTONE 1	M-5
MILLSTONE 2	M-7
MONTICELLO	M-9
NINE MILE POINT 1	N-3
NORTH ANNA 1	N-5
OCONEE 1	O-3
OCONEE 2	O-5
OCONEE 3	O-7
OYSTER CREEK 1	O-9
PALISADES	P-3
PEACH BOTTOM 2	P-5
PEACH BOTTOM 3	P-7
PILGRIM 1	P-9
POINT BEACH 1	P-11
POINT BEACH 2	P-13
PRAIRIE ISLAND 1	P-15
PRAIRIE ISLAND 2	P-17
QUAD CITIES 1	Q-3
QUAD CITIES 2	Q-5
RANCHO SECO 1	R-3
ROBINSON 2	R-5
SALEM 1	S-3
SAN ONOFRE 1	S-5
ST LUCIE 1	S-7
SURRY 1	S-9
SURRY 2	S-11
THREE MILE ISLAND 1	T-3
THREE MILE ISLAND 2	T-5
TROJAN	T-7
TURKEY POINT 3	T-9
TURKEY POINT 4	T-11
VERMONT YANKEE 1	V-3
YANKEE-ROWE 1	Y-3
ZION 1	Z-3
ZION 2	Z-5

SECTION 1 – CURRENT DATA SUMMARIES

MONTHLY HIGHLIGHTS OF COMMERCIAL NUCLEAR POWER UNITS

FOR MONTH OF

JANUARY 1980

LICENSED POWER REACTORS:

NUMBER		CAPACITY[1] MWe (Net)
66	IN COMMERCIAL OPERATION	48,173
1	NOT IN COMMERCIAL OPERATION	912
67[2]	LICENSED TO OPERATE	49,085

POWER GENERATION

	REPORT MONTH	PREVIOUS MONTH	YEAR-TO-DATE
1. GROSS ELECTRICAL (MWHE)	20,039,154	21,468,763	20,039,154
2. NET ELECTRICAL (MWHE)	19,087,131	20,474,669	19,087,131
3. AVG. UNIT SERVICE FACTOR (%)	61.6	66.4	61.6
4. AVERAGE UNIT AVAILABILITY FACTOR (%)	62.0	66.6	62.0
5. AVERAGE UNIT CAPACITY FACTOR (MDC) (%)	54.6	60.7	54.6
6. AVG. UNIT CAPACITY FACTOR (DER) (%)	53.3	59.2	53.3
7. FORCED OUTAGE RATE (%)	8.5	13.0	8.5

NOTES

[1] BASED UPON MAXIMUM DEPENDABLE CAPACITY; DESIGN ELECTRIC RATING USED IF MDC NOT DETERMINED.

[2] EXCLUDES INDIAN POINT 1 (DER = 265), HUMBOLDT BAY (DER = 65), AND THREE MILE ISLAND 2 (DER = 906) WHICH HAVE OPERATING LICENSES BUT ARE SHUTDOWN INDEFINITELY.

ACTUAL VS. POTENTIAL ENERGY PRODUCTION

POTENTIAL ENERGY PRODUCTION IN THIS PERIOD BY UNITS IN COMMERCIAL OPERATION (Using Maximum Dependable Capacity Net) 35,840,712 MWHe

			% OF POTENTIAL PRODUCTION
ENERGY ACTUALLY PRODUCED DURING THIS REPORT PERIOD	=	19,087,131 MWHe NET	53.3
ENERGY NOT PRODUCED DURING THIS REPORT PERIOD DUE TO SCHEDULED OUTAGES (NET)	=	10,589,836 MWHe	29.5
ENERGY NOT PRODUCED DURING THIS REPORT PERIOD DUE TO FORCED OUTAGES (NET)	=	3,628,392 MWHe	10.1
ENERGY NOT PRODUCED DURING THIS REPORT PERIOD FOR OTHER REASONS (NET)	=	2,535,352 MWHe	7.1
TOTAL		35,840,712 MWHe =	100%
ENERGY NOT PRODUCED THIS REPORT PERIOD DUE TO NRC RESTRICTIONS	=	5,088,641 MWHe	14.2
NUMBER OF UNITS WITH POWER RESTRICTIONS IMPOSED BY NRC	=	1	

OUTAGE DATA

	NUMBER	HOURS	PERCENT OF CLOCK TIME	MWHE LOST PRODUCTION
1. FORCED OUTAGES DURING MONTH	44	4,030.2	8.2	3,628,392
2. SCHEDULED OUTAGES DURING THE MONTH	38	14,805.5	30.2	10,589,836
3. TOTAL	82	18,835.7	38.4	14,218,229

MWHE LOST PRODUCTION = Down time x maximum dependable capacity net

DERATED UNITS

UNIT NAME	MAXIMUM DEPENDABLE CAPACITY (MWe - NET)	POWER LIMIT (MWe - NET)
BIG ROCK POINT 1	63	63
DRESDEN 3	773	550
† FORT ST VRAIN	330	231
MAINE YANKEE	810	500
POINT BEACH 1	495	390

†Restricted by NRC

MONTHLY HIGHLIGHTS OF COMMERCIAL NUCLEAR POWER UNITS

FOR MONTH OF

JANUARY 1980

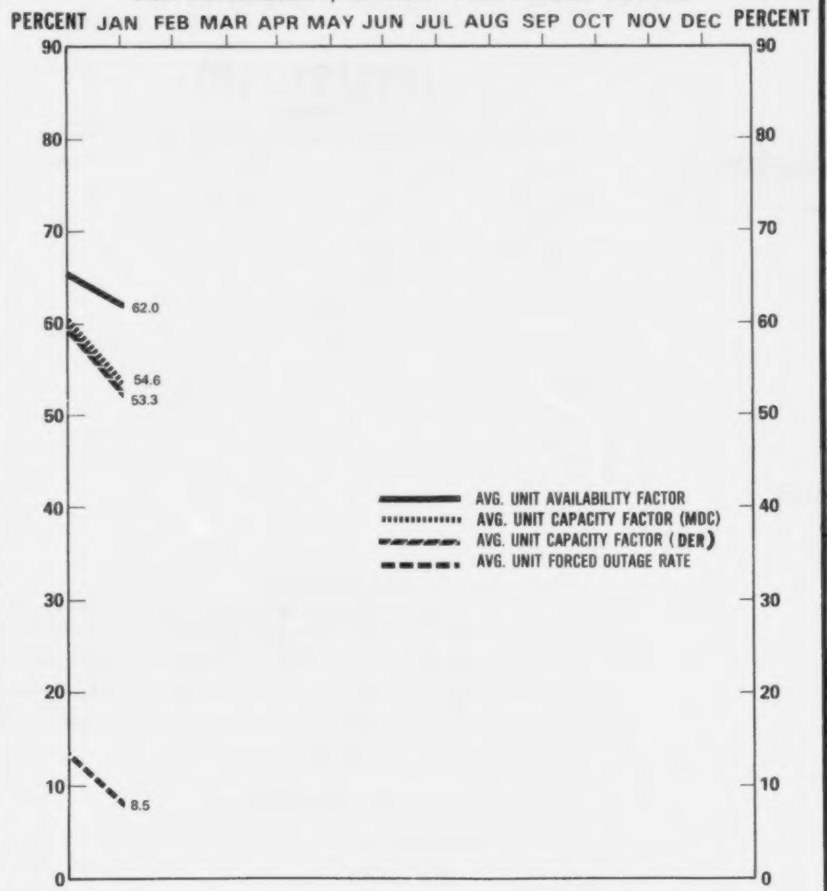
REASONS FOR SHUTDOWN

REASON	NUMBER	TOTAL HOURS LOST
A - Equipment Failure	26	1,478.400
B - Maintenance or Test	12	1,603.200
C - Refueling	15	9,609.900
D - NRC Restriction	17	5,880.100
E - Operator Training and License Examination	0	0
F - Administrative	1	2.000
G - Operational Error	5	37.500
H - Other	6	224.600
TOTAL	82	18,835.7

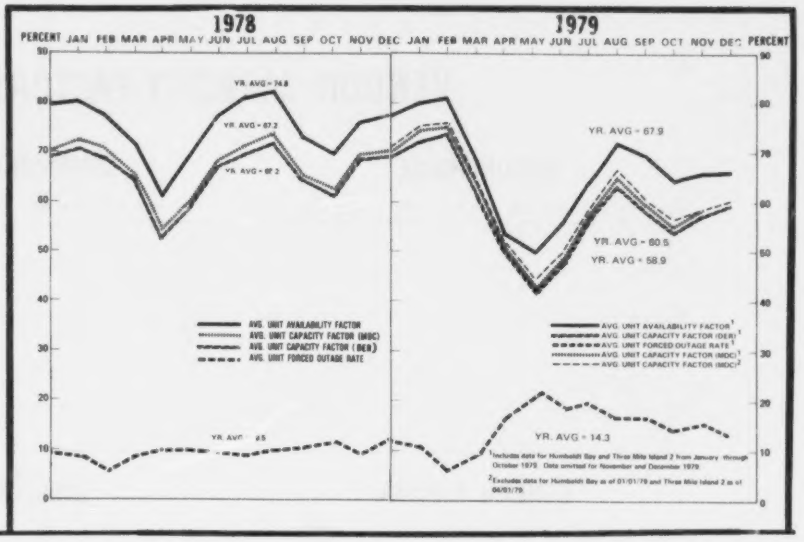
SHUTDOWNS GREATER THAN 72 HOURS EACH

UNIT NAME	REASON	UNIT NAME	REASON
ARKANSAS 1	D	MAINE YANKEE	C
BEAVER VALLEY 1	C	NORTH ANNA 1	C
BIG ROCK POINT 1	D	OCONEE 1	C
BROWNS FERRY 1	C	OYSTER CREEK 1	C
BROWNS FERRY 3	A	PALISADES	D
BRUNSWICK 2	D	PEACH BOTTOM 2	2B
CALVERT CLIFFS 1	D	PEACH BOTTOM 3	B
CALVERT CLIFFS 2	D	PILGRIM 1	C
COOK 1	D	PRAIRIE ISLAND 2	C
COOK 2	D	QUAD CITIES 2	C
DAVIS-BESSE 1	B	RANCHO SECO 1	C
DRESDEN 1	C	SAN ONOFRE 1	H
FORT CALHOUN 1	C	SURRY 1	D
FORT ST VRAIN	B	SURRY 2	C
HATCH 1	A	THREE MILE ISLAND 1	D
HATCH 2	D	TURKEY POINT 3	C
INDIAN POINT 2	D	YANKEE-ROWE 1	D
INDIAN POINT 3	C	ZION 1	A,D
KEWAUNEE	2A	ZION 2	D

UNIT AVAILABILITY, CAPACITY AND FORCED OUTAGE



UNIT AVAILABILITY, CAPACITY AND FORCED OUTAGE

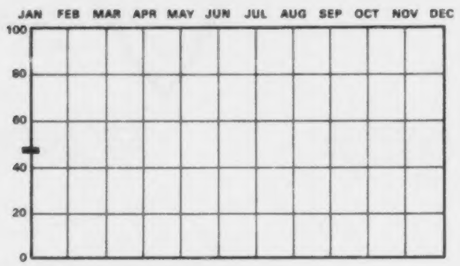


REFUELING SCHEDULE

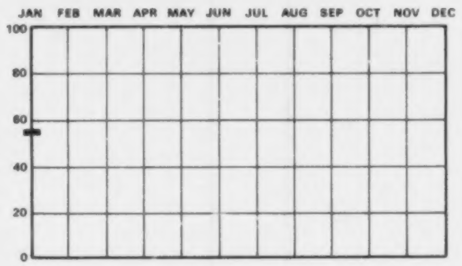
Please refer to the table on page 3–6 for the scheduled refueling date for each nuclear powerplant in this publication.

VENDOR CAPACITY FACTORS

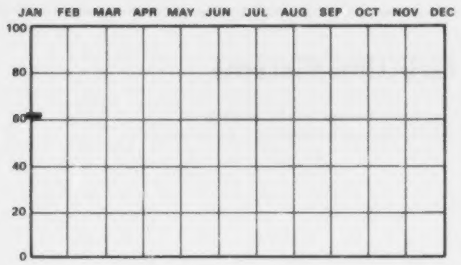
WESTINGHOUSE



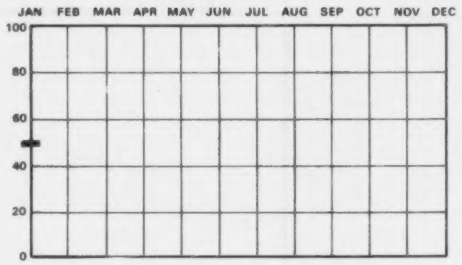
COMBUSTION ENGINEERING



GENERAL ELECTRIC



BABCOCK & WILCOX



JANUARY 1980

OPERATIONS

1. COMPUTED BY ARITHMETIC AVERAGE OF UNIT CAPACITY FACTORS

GE BWR's	CFMDC
BROWNS FERRY 1	7.2
BROWNS FERRY 2	87.0
BROWNS FERRY 3	37.1
BRUNSWICK 1	90.6
BRUNSWICK 2	67.7
COOPER STATION	84.8
DRESDEN 1	.0
DRESDEN 2	89.0
DRESDEN 3	66.9
DUANE ARNOLD	84.1
FITZPATRICK	94.1
HATCH 1	71.4
HATCH 2	83.3
MILLSTONE 1	44.6
MONTICELLO	32.8
NINE MILE POINT 1	95.1
OYSTER CREEK 1	10.5
PEACH BOTTOM 2	32.5
PEACH BOTTOM 3	71.9
PILGRIM 1	11.8
QUAD CITIES 1	95.1
QUAD CITIES 2	.0
VERMONT YANKEE 1	90.0

23 UNITS

W PWR's	CFMDC
BEAVER VALLEY 1	.0
COOK 1	40.1
COOK 2	21.6
FARLEY 1	85.4
GINNA	101.3
HADDAM NECK	105.1
INDIAN POINT 2	26.7
INDIAN POINT 3	.0
KEWAUNEE	57.7
NORTH ANNA 1	6.7
POINT BEACH 1	75.0
POINT BEACH 2	96.0
PRAIRIE ISLAND 1	96.5
PRAIRIE ISLAND 2	4.4
ROBINSON 2	101.5
SALEM 1	69.3
SAN ONOFRE 1	72.8
SURRY 1	70.9
SURRY 2	.0
TROJAN	92.9
TURKEY POINT 3	.0
TURKEY POINT 4	91.5
YANKEE-ROWE 1	58.2
ZION 1	.0
ZION 2	31.0

25 UNITS

CE PWR's	CFMDC
CALVERT CLIFFS 1	41.6
CALVERT CLIFFS 2	70.6
FORT CALHOUN 1	53.2
MAINE YANKEE	21.7
MILLSTONE 2	99.2
PALISADES	0.0
ST LUCIE 1	96.3

7 UNITS

B&W PWR's	CFMDC
ARKANSAS 1	.0
CRYSTAL RIVER 3	96.4
DAVIS-BESSE 1	73.7
OCONEE 1	.0
OCONEE 2	93.0
OCONEE 3	98.2
RANCHO SECO 1	31.8
THREE MILE ISLAND 1	.0

8 UNITS

2. COMPUTED BY RATIO OF VENDOR TOTALS

	GE BWR's	W PWR's	CE PWR's	B&W PWR's	All PWR's
NET ELECTRICAL PRODUCTION	7,857,163	6,462,254	2,189,881	2,495,423	11,147,558
MDC NET	17,351	18,435	5,178	6,768	30,381
CFMDC	61.1	47.1	56.8	49.6	49.3

Capacity factor in this page, denoted as CFMDC, is a function of the net maximum dependable capacity. See the corresponding definition in the Glossary. The vendor averages are computed by the formula:

$$\frac{\text{Net Electrical Energy Produced by Vendor}}{\text{Potential Electrical Production By Vendor In This Month}} \times 100\%$$

Units Excluded	Reason for Exclusion
Indian Point 1 Humboldt Bay Three Mile Island 2	Unit Shutdown Indefinitely
Arkansas 2	Not in Commercial Operation
Big Rock Point	Not Representative of Typical BWRs
LaCrosse	Not Representative of Typical BWRs and Atypical Vendor
Fort St. Vrain	Atypical Vendor and Reactor Design

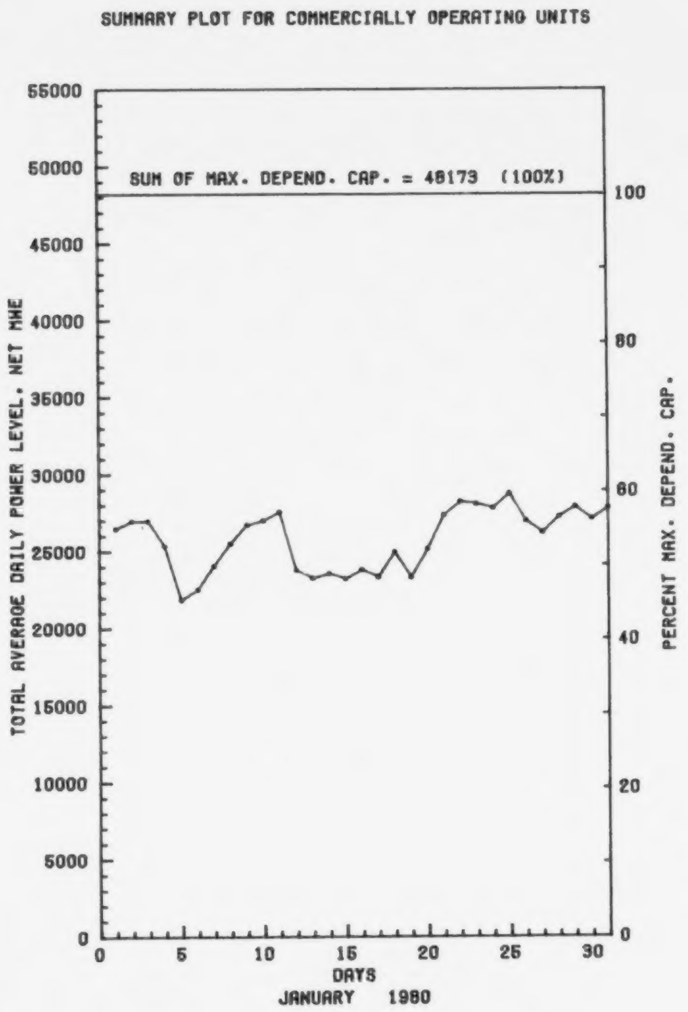
AVERAGE DAILY POWER LEVEL FOR ALL COMMERCIALLY OPERATING UNITS

This chart depicts the average daily power level for the units in commercial operation during the month.

The straight line on the graph labelled "SUM OF MAX DEP CAP'S" is plotted at the value shown by summing the separate maximum dependable capacities of the commercially operating units (in Net MWe). The plot shown below the line is calculated by summing the separate average daily power levels of the commercially operating units for each day of the month.

The scale on the left vertical axis runs in 1,000 MWe increments from 0 to 55,000 MWe (Net). The right vertical axis shows the percentage in 10% increments, up to 100% of the "SUM OF MAX DEP CAP'S."

It should be recognized that the 100% line would be obtainable only if all of the commercially operating units operated at 100% capacity, 24 hours per day, for the entire month. In other words, since any power generator must occasionally shut down to refuel and/or perform needed maintenance, and also since 100% capacity production is not always required by power demands, the 100% line is a theoretical goal and not a practical one.



ERRATA

Corrections and Amendments to Previously-Reported Data

(NOTE: These changes are reflected in the data contained in the current report)

CORRECTIONS TO NUREG 0020, VOL. 4 NO. 1, JANUARY 1980

Page	Unit Name	Item	Correction
C-13	Crystal River 3	16. Gross Thermal Energy (MWH)	The monthly value for December 1979 should have been 588,283 rather than the published value of 588,258 MWt.

MULTIPLE - ISSUE CORRECTIONS

BEAVER VALLEY	THE LICENSED THERMAL POWER SHOULD HAVE BEEN LISTED AS 2660 MWt RATHER THAN THE PUBLISHED VALUE OF 2652 MWt.
NINE MILE POINT	THE LICENSEE HAS RECOMPUTED THE LIFETIME VALUE OF THE GROSS THERMAL ENERGY'. MONTHLY CHANGES (IF AFTER 12/78) WILL BE PUBLISHED WHEN FURTHER INFORMATION IS RECEIVED FROM THE LICENSEE.
RANCHO SECO	THE LIFETIME VALUE OF GROSS THERMAL ENERGY HAS BEEN RECOMPUTED DUE TO A CHANGE IN THE CALCULATING COEFFICIENT. THE REVISED MONTHLY VALUES ARE:

DECEMBER 78	573,006 MWt
JANUARY 79	1,924,056
FEBRUARY 79	1,634,206
MARCH 79	2,028,439
APRIL 79	1,718,884
MAY 79	1,453,504
JUNE 79	N/A
JULY 79	N/A
AUGUST 79	1,991,649
SEPTEMBER 79	1,834,975
OCTOBER 79	1,745,162
NOVEMBER 79	1,615,034

SECTION 2 – OPERATING POWER REACTORS

ARKANSAS 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Arkansas/Pope 6 miles NW of Russelville
2.	Electric Reliability Council	Member of SW Power Pool
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2568
5.	Design Electrical Rating (MWE Net)	850
6.	Date of Initial Criticality	August 6, 1974
7.	Date of Commercial Operation	December 19, 1974
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Dardenalle Reservoir

Utility & Contractor Information

10.	Licensee	Arkansas Power and Light Company
11.	Corporate Address	P.O. Box 551 Little Rock, Arkansas 72203
12.	Corporate Contact	W. Cavanaugh III Vice Pres., Generation & Construction
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	B&W
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	IV
17.	IE Branch Chief	G. L. Madsen
18.	IE Senior Inspector	T. F. Westerman
19.	IE Principal Inspector	W. D. Johnson
20.	Licensing Project Manager	G. S. Vissing
21.	Docket Number	50-313
22.	License Number/Date of Issue	DPR-51, 5/21/74
23.	Public Document Room	Arkansas Polytechnic College Russelville, Arkansas 72801

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO TECHNICAL SPECIFICATION 6.8.1, THE LICENSEE FAILED TO MAINTAIN PROCEDURE 1203.12, ANNUNCIATOR CORRECTIVE ACTION, SECTION K11,E-2, IN THAT THIS PROCEDURE WAS NOT REVISED AS REQUIRED FOLLOWING THE IMPLEMENTATION OF AMENDMENT 39 TO THE TECHNICAL SPECIFICATIONS. (7925 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

FACILITY ITEMS (PLANS AND PROCEDURES):

MANAGERIAL ITEMS

PLANT STATUS:

+ THE UNIT WAS SHUT DOWN DURING THE MONTH OF JANUARY FOR LOW PRESSURE TURBINE REPAIRS, PIPE SUPPORT MODIFICATIONS, AND DESIGN CHANGES REQUIRED BY NUREG 0578.

LAST IE SITE INSPECTION DATE: JANUARY 7 - JANUARY 18, 1980 +

INSPECTION REPORT NO: 50-313/80-01 +

SUMMARY

INSPECTION SUMMARY

INSPECTION CONDUCTED DURING PERIOD OF NOVEMBER 22 - DECEMBER 21, 1979 (79-25): ROUTINE, ANNOUNCED INSPECTION OF PREVIOUSLY IDENTIFIED INSPECTION ITEMS, LICENSEE ACTIONS IN RESPONSE TO IE BULLETINS, PLANT OPERATIONS, AND PROCEDURES. WITHIN THE FOUR AREAS INSPECTED, ONE ITEM OF NONCOMPLIANCE WAS IDENTIFIED (INFRACTION - FAILURE TO MAINTAIN PROCEDURE).

INSPECTION CONDUCTED DURING THE PERIOD OF DECEMBER 17-20, 1979 (79-26): ROUTINE, UNANNOUNCED INSPECTION OF PREVIOUSLY IDENTIFIED ITEMS, CALIBRATION OF LICENSE SPECIFIED INSTRUMENTATION AND CALIBRATION OF GENERAL SAFETY-RELATED INSTRUMENTATION. NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION CONDUCTED DURING THE PERIOD OF DECEMBER 18-20, 1979 (79-27): INCLUDED A REVIEW OF: SECURITY PLAN; SECURITY ORGANIZATION; THE PROTECTED AREA PHYSICAL BARRIERS; ACCESS CONTROL (IDENTIFICATION, AUTHORIZATION, AND BADGING); FOLLOW-UP ON PREVIOUS UNRESOLVED ITEMS. THE LICENSEE WAS FOUND TO BE IN COMPLIANCE WITH NRC REQUIREMENTS WITHIN ALL AREAS EXAMINED DURING THIS INSPECTION EXCEPT FOR THE FOLLOWING. THE LICENSEE FAILED TO ENFORCE THE ISP REQUIREMENT THAT ALL VEHICLES (LDV) HAVE THE IGNITION KEY REMOVED AND THE LDV CAB LOCKED.

INSPECTION CONDUCTED JANUARY 7 - 18, 1980 (80-01): ROUTINE, UNANNOUNCED INSPECTION OF PREVIOUSLY IDENTIFIED ITEMS, AND CONTROL OF DESIGN CHANGES AND MODIFICATIONS. TWO ITEMS OF NONCOMPLIANCE INVOLVING FAILURE TO COMPLY WITH APPENDIX B TO 10 CFR 50 WERE IDENTIFIED.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

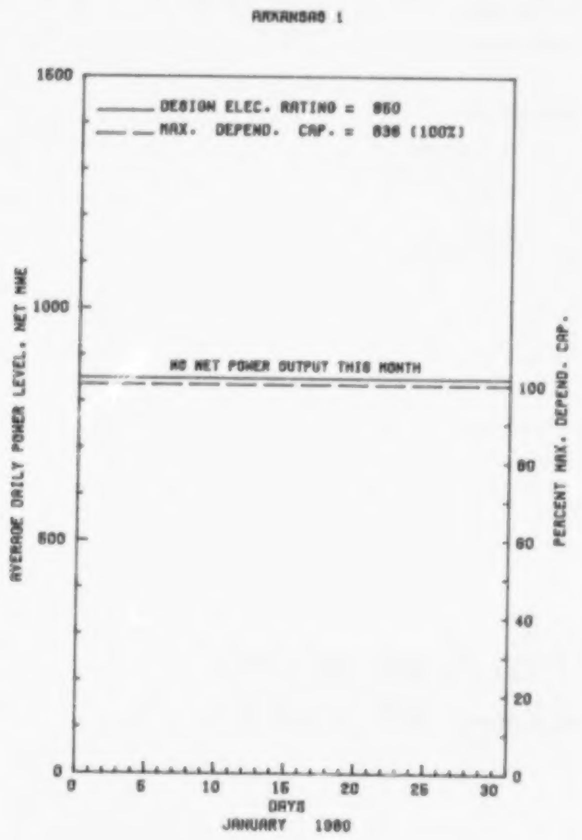
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
79-23 03L-0	12/21/79	1/18/80	AUTOMATIC WITHDRAWAL OF CONTROL RODS ABOVE SPECIFIED LIMITS
79-24 03L-0	12/6/79	1/10/80	INOPERABLE STEAM DRIVEN EMERGENCY FEEDWATER PUMP.

REVIEWED BY OMPA LOEB: John J Rizzo **DATE:** 3-19-80

REVIEWED BY NRR DOR: G.S. Vissing **DATE:** 3-20-80

ARKANSAS 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-313
2. Utility Contact: L. S. Bramlett (501) 968-2519
3. Licensed Thermal Power (MWt): 2568
4. Nameplate Rating (Gross MWe): 1003 x 0.9 = 903
5. Design Electrical Rating (Net MWe): 850
6. Maximum Dependable Capacity (Gross MWe): 883
7. Maximum Dependable Capacity (Net MWe): 836
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	744.0	44,875.0
12. Hours Reactor Was Critical	.0	.0	30,138.2
13. Reactor Reserve Shutdown Hours	.0	.0	4,247.4
14. Hours Generator On-Line	.0	.0	29,470.5
15. Unit Reserve Shutdown Hours	.0	.0	796.7
16. Gross Thermal Energy (MWH)	0	0	71,262,044
17. Gross Electrical Energy (MWH)	0	0	23,721,156
18. Net Electrical Energy (MWH)	0	0	22,634,036
19. Unit Service Factor	.0	.0	65.7
20. Unit Availability Factor	.0	.0	67.4
21. Unit Capacity Factor (MDC Net)	.0	.0	60.3
22. Unit Capacity Factor (DER Net)	.0	.0	59.3
23. Unit Forced Outage Rate	.0	.0	15.6

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 02/09/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
79-06	12/31/79	S	744.0	D	4		CH		INSTRU COMMITTMENT TO NRC TO PROVIDE TMI II RELATED MODIFICATIONS.

SUMMARY

THE UNIT REMAINED SHUTDOWN THROUGHOUT THE REPORT PERIOD TO MAKE TMI-RELATED MODIFICATIONS.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

ARKANSAS 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Arkansas/Pope 6 miles NW of Russelville
2.	Electric Reliability Council	Member of SW Power Pool
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2815
5.	Design Electrical Rating (MWE Net)	912
6.	Date of Initial Criticality	12-5-78
7.	Date of Commercial Operation	N/A
8.	Condenser Cooling Method	Recirculating
9.	Condenser Cooling Water Source	Cooling Tower

Utility & Contractor Information

10.	Licensee	Arkansas Power & Light Company
11.	Corporate Address	P.O. Box 551 Little Rock, Arkansas 72203
12.	Corporate Contact	W. Cavanaugh III Vice Pres., Generation & Construction
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Combustion Engineering, Inc.
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	IV
17.	IE Branch Chief	G. L. Madsen
18.	IE Senior Inspector	T. F. Westerman
19.	IE Principal Inspector	W. D. Johnson
20.	Licensing Project Manager	G. S. Vissing
21.	Docket Number	50-368
22.	License Number/Date of Issue	NFP-6, 9/1/78
23.	Public Document Room	Arkansas Polytechnic College Russelville, Arkansas 72801

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO TECHNICAL SPECIFICATION 6.8.1, THE LICENSEE FAILED TO ADHERE TO THE PROCEDURAL REQUIREMENTS OF PROCEDURE 2.300.01, POWER ASCENSION TEST, APPENDIX U, ATTACHMENT U-5, REACTOR REGULATING SYSTEM TEST, STEPS B.5, C.5, B.10 AND C.10 WITH RESPECT TO THE METHOD PRESCRIBED TO BE USED FOR CHANGING AVERAGE REACTOR COOLANT SYSTEM TEMPERATURE.
(7924 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

The 4 RTD's on the west side of the hot legs normally indicate a temperature about 3 degrees F higher than the 4 RTD's on the east side of the hot legs at 50 % power. This relationship occasionally reverses. A design change to minimize the effect of the anomaly on the Core Protection Calculators has been installed.

FACILITY ITEMS (PLANS AND PROCEDURES):

MANAGERIAL ITEMS:

PLANT STATUS:

DURING THIS PERIOD THE UNIT COMPLETED 80% PLATEAU AND PART OF 100% PLATEAU OF POWER ASCENSION TEST. THE PLANT IS NOW IN COLD SHUTDOWN IMPLEMENTING NUREG 0578 REQUIREMENTS.

LAST IE INSPECTION DATE: JANUARY 7 - 18, 1980

INSPECTION REPORT NO: 50-368/80-01

SUMMARY

INSPECTION SUMMARY

INSPECTION CONDUCTED DURING PERIOD OF NOVEMBER 22 - DECEMBER 21, 1979 (79-24): ROUTINE, ANNOUNCED INSPECTION OF PREVIOUSLY IDENTIFIED INSPECTION ITEMS, POWER ASCENSION TESTING, 50% PLATEAU DATA REVIEW, FOLLOW-UP ON LICENSEE EVENT REPORTS, AND LICENSEE ACTION IN RESPONSE TO IE BULLETINS. ONE APPARENT ITEM OF NONCOMPLIANCE INVOLVING FAILURE TO ADHERE TO PROCEDURE WAS IDENTIFIED.

INSPECTION CONDUCTED DURING THE PERIOD OF DECEMBER 17-20, 1979 (79-25): ROUTINE, UNANNOUNCED INSPECTION OF PREVIOUSLY IDENTIFIED ITEMS, CALIBRATION OF LICENSE SPECIFIED INSTRUMENTATION AND CALIBRATION OF GENERAL SAFETY-RELATED INSTRUMENTATION. NO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED.

INSPECTION CONDUCTED DURING THE PERIOD OF DECEMBER 18-20, 1979 (79-26) INCLUDED A REVIEW OF: SECURITY PLAN; SECURITY ORGANIZATION; THE PROTECTED AREA PHYSICAL BARRIERS; ACCESS CONTROL (IDENTIFICATION, AUTHORIZATION, AND BADGING); FOLLOW-UP ON PREVIOUS UNRESOLVED ITEMS. ONE APPARENT ITEM OF NONCOMPLIANCE INVOLVING FAILURE TO ENFORCE THE ISP REQUIREMENT THAT ALL VEHICLES HAVE IGNITION KEY REMOVED AND THE LDV CAB LOCKED WAS IDENTIFIED.

INSPECTION CONDUCTED JANUARY 7-18, 1980 (80-01): ROUTINE, UNANNOUNCED INSPECTION OF PREVIOUSLY IDENTIFIED ITEMS, AND CONTROL OF DESIGN AND DESIGN CHANGES AND MODIFICATIONS. TWO APPARENT ITEMS OF NONCOMPLIANCE INVOLVING FAILURE TO COMPLY WITH APPENDIX B TO 10 CFR 50 WERE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

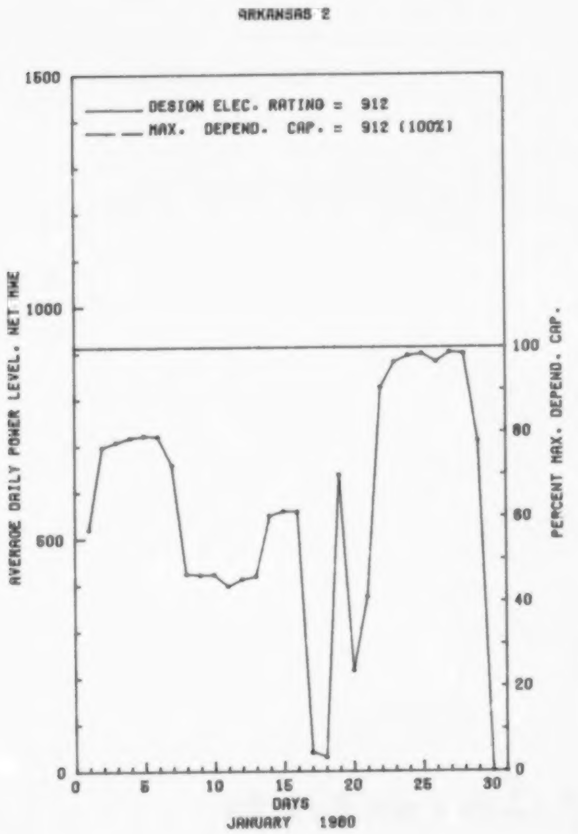
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
79-88 03L-0	11/22/79	12/21/79	FAILURE OF EFW CONTROL VALVE 2CV-1025-1
79-89 03L-0	11/24/79	12/21/79	FAILURE OF EFW CONTROL VALVE 2CV-1025-1
79-90 03L-0	11/25/79	12/21/79	FAILURE OF EFW CONTROL VALVE 2CV-1025-1
79-91 03L-0	11/26/79	12/21/79	HIGH O2 IN RCS
79-92 03L-0	12/1/79	12/21/79	FAILURE OF EFW CONTROL VALVE 2CV-1025-1
79-94 03L-0	12/7/79	1/4/80	FAILURE OF CH 'B' PER PRESS VARIABLE SETPOINT
79-95 03L-0	12/11/79	1/4/80	BATTERY CHARGER FAILURE
79-96 03L-0	12/11/79	1/4/80	CPC CHANNEL FAILURE
79-97 03L-0	11/28/79	12/28/79	CONTAINMENT BUILDING SUMP LEVEL IND. FAILURE
79-98 03L-0	12/5/79	1/9/80	CPC CHANNEL FAILURE
79-99 03L-0	12/5/79	1/9/80	PERSONNEL AIRLOCK DOOR OPEN
79-101 03L-0	12/17/79	1/16/80	FAILURE OF RWT LEVEL INDICATOR
79-102 03L-0	12/21/79	1/18/80	CPC CHANNEL 'C' FAILURE
79-103 03L-0	12/30/79	1/29/80	PLANT PROTECTIVE SYSTEM CHANNEL FAILURE
79-104 03L-0	12/30/79	1/29/80	STEAM DRIVEN EFW PUMP OVERSPEED TRIP

REVIEWED BY OMPA LOEB: Beebe **DATE:** 03/19/80

REVIEWED BY NRR DOR: Guy S Vissing **DATE:** 3/20/80

ARKANSAS 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-368
2. Utility Contact: Rex Pendergraft (501) 968-2519
3. Licensed Thermal Power (MWt): 2815
4. Nameplate Rating (Gross MWe): 959
5. Design Electrical Rating (Net MWe): 912
6. Maximum Dependable Capacity (Gross MWe): 912
7. Maximum Dependable Capacity (Net MWe): 912
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	744.0	10,248.0
12. Hours Reactor Was Critical	691.0	691.0	4,519.4
13. Reactor Reserve Shutdown Hours	.0	.0	2,293.1
14. Hours Generator On-Line	628.1	628.1	3,665.0
15. Unit Reserve Shutdown Hours	.0	.0	21.7
16. Gross Thermal Energy (MWH)	1,322,966	1,322,966	4,819,777
17. Gross Electrical Energy (MWH)	434,827	434,827	1,418,112
18. Net Electrical Energy (MWH)	410,234	410,[illegible]	1,296,043
19. Unit Service Factor			
20. Unit Availability Factor			
21. Unit Capacity Factor (MDC Net)			
22. Unit Capacity Factor (DER Net)			
23. Unit Forced Outage Rate			

NOT IN COMMERCIAL OPERATION

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: MARCH, 1980

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-01	01/17/80	F	42.1	A	3		HH		FEEDWATER CONTROL VALVE FAILED DUE TO LOSS OF AIRLINE. AIRLINE WAS REPLACED.
80-02	01/19/80	F	14.5	A	3		HA	TURBIN	TURBINE GENERATOR TRIPPED DUE TO HIGH VIBRATION.
80-03	01/21/80	F	6.3	A	3		HA	TURBIN	SECOND TRIP DUE TO HIGH VIBRATION.
80-04	01/29/80	S	53.0	B	2				SCHEDULED REACTOR/TURBINE TRIP PER STARTUP TESTING SEQUENCE.

SUMMARY

THE UNIT CONTINUED THE STARTUP TESTING PROGRAM THROUGHOUT THE REPORT PERIOD. PROBLEMS OCCURRED WITH FEEDWATER CONTROL, A PRESSURIZER SPRAY VALVE, AND HIGH VIBRATION ON THE MAIN TURBINE GENERATOR.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

BEAVER VALLEY 1

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Pennsylvania/Beaver 5 miles E of E. Liverpool, Ohio
2.	Electric Reliability Council	ECAR
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2660
5.	Design Electrical Rating (MWE Net)	852
6.	Date of Initial Criticality	May 10, 1976
7.	Date of Commercial Operation	10/01/76 (50%) 04/30/77 (100%)
8.	Condenser Cooling Method	Cooling tower/Closed loop
9.	Condenser Cooling Water Source	Ohio River

Utility & Contractor Information

10.	Licensee	Duquesne Light Company
11.	Corporate Address	435 Sixth Avenue Pittsburgh, Pennsylvania 15219
12.	Corporate Contact	C. N. Dunn, Vice President
13.	Architect/Engineer	Stone & Webster
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Senior Inspector	R. R. Keimig
19.	IE Principal Inspector	D. Beckman
20.	Licensing Project Manager	D. L. Wigginton
21.	Docket Number	50-334
22.	License Number/Date of Issue	DPR-66, 1/30/76
23.	Public Document Room	B. F. Jones Memorial Library 633 Franklin Avenue Aliquippa, PA 15001

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO 10 CFR 55 APPENDIX A & THE BV TRAINING MANUAL, 6 INDIVIDUALS WERE DELINQUENT IN INDIVIDUAL STUDY ASSIGNMENTS & 1 INDIVIDUAL WAS DELINQUENT IN REMEDIAL TRAINING.
(7923 DEFICIENCY)

NONCONTROLLED COPIES OF SURVEILLANCE PROCEDURES WERE BEING MAINTAINED BY SHIFT SUPERVISOR CONCURRENTLY WITH CONTROLLED COPIES OF THE SAME PROCEDURES. THIS CAUSED SOME OF THE CONTROLLED COPIES NOT TO HAVE THE LATEST CHANGE POSTED AND TO HAVE AT LEAST ONE SURVEILLANCE PROCEDURE TO HAVE BEEN ACCOMPLISHED WITHOUT THE LATEST CHANGE POSTED.
(7925 DEFICIENCY)

TS 6.11 REQUIRES THAT "PROCEDURES FOR PERSONNEL RADIATION PROTECTION SHALL BE PREPARED CONSISTENT WITH THE REQUIREMENTS OF 10 CFR PART 20 & SHALL BE APPROVED, MAINTAINED & ADHERED TO FOR ALL OPERATIONS INVOLVING PERSONNEL RADIATION EXPOSURE". CONTRARY TO THE ABOVE REQUIREMENT, DURING THE PERIOD 11/21-23/79, ON 3 OCCASIONS, RESPIRATORS WERE ISSUED TO PERSONNEL WHO WERE NOT ON THE TRAINING LIST & ON 1 OCCASION A RESPIRATOR WAS ISSUED TO AN INDIVIDUAL WHO WAS NOT LISTED FOR THAT TYPE OF RESPIRATOR.
(7926 INFRACTION)

CONTRARY TO TS 6.8.1 & STATION ADMINISTRATIVE DIRECTIVE 08, CORRECTIVE MAINTENANCE PROCEDURE NO. 1-75-159, "OPERATING & MAINTENANCE PROCEDURE FOR ITT GRINNELL SNUBBER TESTER" WAS NOT IMPLEMENTED IN THAT: DURING THE FUNCTIONAL TESTING OF ITT GRINNELL SNUBBERS FROM 7/5 TO 7/13/79 THE HYDRAULIC FLUID TEMPERATURE WAS NOT MEASURED OR RECORDED ON THE TEST REPORT FORM PHD-5434-F.
(7929 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

None.

FACILITY ITEMS (PLANS AND PROCEDURES):

+ The facility began an extended refueling/modification outage on 11/30/79.

MANAGERIAL ITEMS:

+ A partial reorganization of station and corporate departments began on 12/16/79, and is substantially complete, pending further personnel assignments and approval of revised Tech. Specs.

PLANT STATUS:

+ Plant was defueled for in-service inspection while continuing to maintain Mode C (refueling) conditions. Core reload is tentatively scheduled to begin 2/14/80.

+ LAST IE SITE INSPECTION DATE: 2/11/80

+ INSPECTION REPORT NO: 50-334/80-01

SUMMARY

INSPECTION SUMMARY

+ 50-334/79-22 - Oct 2-Nov 2: Routine resident inspector review (55 hours onsite). Inspection areas (8): Action on previous inspection findings; plant operations; IE Bulletin and Circular followup; plans for coping with strikes; licensee event reports; periodic reports; and followup on a reactor trip. Noncompliances: None.

+ 50-334/79-25 - Nov 13-16: Routine, unannounced inspection by a region-based inspector (30 hours onsite). Inspection areas (5): administrative controls for surveillance procedures; surveillance testing; inspector witnessing of surveillance testing; technician qualification; and control room observations. Noncompliances: One (Failure to maintain controlled copies of surveillance procedures current).

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-002SP	07/17/79	01/11/80	Other	Fire main system rupture
79-039/ 99X	09/20/79	12/19/79	Special	No. 4 vital bus inverter failed resulting in a reactor trip and a safety injection
79-040/ 99X	09/20/79	12/19/79	Special	While in Mode 3, No. 4 vital bus inverter was transferred from its auxiliary power source to the deenergized inverter resulting in a safety injection
79-045/ 03L	11/08/79	12/07/79	30 Day LER	Boric acid storage tank concentration less than required
79-046/ 01T	11/27/79	12/11/79	24 Hours	RWST suctions to HPSI inop for 2 hr for auto si initiation on loss of offsite power
80-001/ 01T	08/03/79	01/04/80	14 Day LER	Pipe systems less than 6 inches found to have been analyzed with improper code and being reanalyzed using "nupipe" program

+ 50-334/79-26 - Nov 26-29: Special, unannounced inspection by a region-based inspector (33 hours). Inspection areas (9): licensee's response to IE Bulletin 79-19; radiation protection during operation and refueling; action on previous inspection findings; document control; training; respiratory protection; licensee audit; procedures; radioaction waste solidification and packaging; refueling preparations; and plant tours. Noncompliances: One (Failure to follow respiratory protection procedures).

+ 50-334/79-29 - Dec 12-14: Routine, unannounced inspection by two region-based inspectors (40 hours). Inspection areas (3): pipe support and restraint systems surveillance; Inservice Inspection Program for Class 1 and 2 restraints; and action on previous inspection findings. Noncompliances: One (Failure to follow procedures).

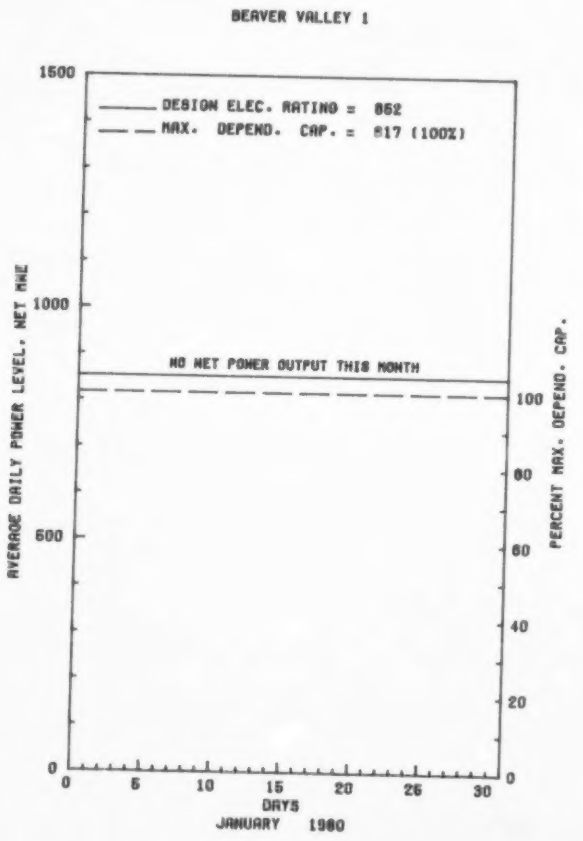
+ 50-334/79-30 - Nov 27-Dec 8: Special inspection by resident inspector (22.5 hours). Inspection areas (1): review of the November 27, 1979 event involving isolation of the High Head Safety Injection Pump suction from the refueling water storage tank. Noncompliances: One (Both independent ECCS subsystems inoperable).

REVIEWED BY OMPA LCEB: T. C. Cintula DATE: 3·19·80

REVIEWED BY NRR DOR: D L Wigginton DATE: 3-21-80

BEAVER VALLEY 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-334
2. Utility Contact: A. E. Spitznogle (412) 643-9087
3. Licensed Thermal Power (MWt): 2660
4. Nameplate Rating (Gross MWe): 1026 x 0.9 = 923
5. Design Electrical Rating (Net MWe): 852
6. Maximum Dependable Capacity (Gross MWe): 845
7. Maximum Dependable Capacity (Net MWe): 817
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	744.0	32,904.0
12. Hours Reactor Was Critical	.0	.0	13,774.7
13. Reactor Reserve Shutdown Hours	.0	.0	4,482.7
14. Hours Generator On-Line	.0	.0	13,105.0
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	0	26,974,253
17. Gross Electrical Energy (MWH)	0	0	8,277,940
18. Net Electrical Energy (MWH)	0	0	7,570,743
19. Unit Service Factor	.0	.0	43.1
20. Unit Availability Factor	.0	.0	43.1
21. Unit Capacity Factor (MDC Net)	.0	.0	33.5
22. Unit Capacity Factor (DER Net)	.0	.0	32.1
23. Unit Forced Outage Rate	.0	.0	46.4

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 07/22/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
1	12/01/79	S	744.0	C	4		ZZ	ZZZZZZ	UNIT SHUTDOWN FOR MAJOR MODIFICATIONS AND REFUELING OUTAGE. STARTUP SCHEDULED FOR JULY, 1980.

SUMMARY

THE UNIT REMAINED SHUTDOWN THROUGHOUT THE REPORT PERIOD FOR MAJOR MODIFICATIONS AND REFUELING. STARTUP IS SCHEDULED FOR JULY 1980.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

Exhibit H: Same source.

BIG ROCK POINT 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Michigan/Charlevoix 4 miles NE of Charlevoix
2.	Electric Reliability Council	ECAR
3.	Type of Reactor	BWR
4.	Power Level (MWT)	240
5.	Design Electrical Rating (MWE Net)	72
6.	Date of Initial Criticality	September 27, 1962
7.	Date of Commercial Operation	March 29, 1963
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10.	Licensee	Consumers Power Company
11.	Corporate Address	212 West Michigan Avenue Jackson, Michigan 49201
12.	Corporate Contact	R. B. DeWitt
13.	Architect/Engineer	Bechtel Corporation
14.	Nuclear Steam Supply System	GE
15.	Constructor	Bechtel Corporation

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Senior Inspector	D.C. Boyd
19.	IE Principal Inspector	K. R. Ridgway
20.	Licensing Project Manager	R. D. Silver
21.	Docket Number	50-155
22.	License Number/Date of Issue	DPR-6 issued 8/30/62
23.	Public Document Room	Charlevoix Public Library 107 Clinton Street Charlevoix, Michigan

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

MANAGERIAL ITEMS

+ Training for 8 of approximately 70 RWP exempt personnel concerning a high radiation occurrence at the fuel pool on January 18, 1979 had not been completed. (79-17)

PLANT STATUS

+ Currently operating at 208 Mwt. The plant was shutdown during the first two weeks of January to make required short term lessons learned changes of NUREG 0578.

+LAST IE SITE INSPECTION DATE: December 4-7, 1979

+ INSPECTION REPORT NO: 79-17

SUMMARY

INSPECTION SUMMARY

+ Inspection on December 4-7, (79-17): Special, announced inspection of solid waste handling, including matters addressed in IE Bulletin No. 79-19 and licensee followup of previous inspection findings. The inspection involved 31 inspector hours onsite by one NRC inspector. No items of noncompliance were noted.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-28/ 01T-0	12/13/79	12/26/79	14 Day LER	Design review of the 24 inch containment ventilation isolation valves, CV4097 and CV4095, on December 13, 1979 indicated that the valves might not close under the worst LOCA pressure conditions in containment unless the valves were operated at partially closed positions.
79-29/ 03L-0	12/15/79	01/15/80	30 Day LER	Neither plant ventilation exhaust fan was operating during an 8-min. planned maintenance activity.

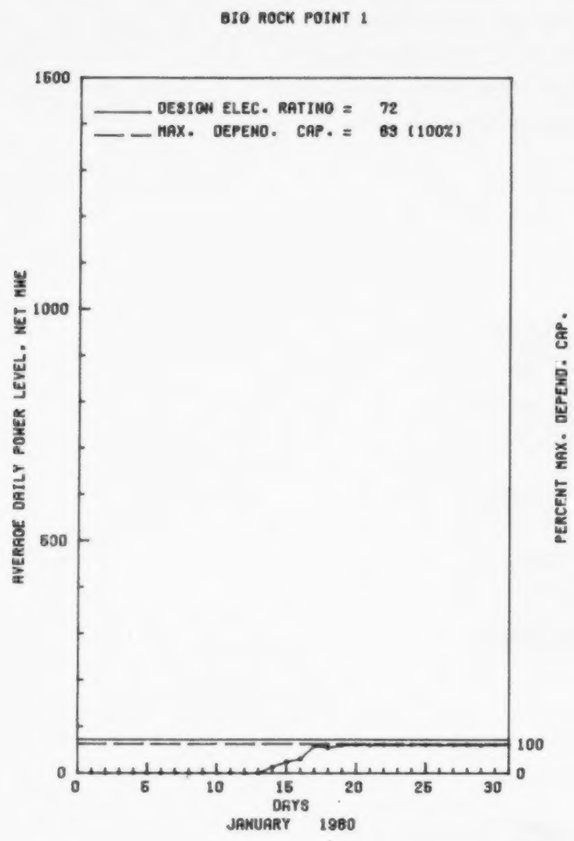
REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 3-19-80

REVIEWED BY NRR DOR: R. D. Silver **DATE:** 3/21/80

GPO 871-667

BIG ROCK POINT 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

ITEM 21 CALCULATED
BY USING A WEIGHTED
AVERAGE

1. Docket Number: 50-155
2. Utility Contact: G. Mimebauch (616) 547-6537 x180
3. Licensed Thermal Power (MWt): 240
4. Nameplate Rating (Gross MWe): 70.6 x 0.85 = 60
5. Design Electrical Rating (Net MWe): 72
6. Maximum Dependable Capacity (Gross MWe): 67
7. Maximum Dependable Capacity (Net MWe): 63
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
MDC NET DECREASED TO 63-FUEL THERMAL HYDRAU. LIMIT MARGINS
9. Power Level To Which Restricted, If Any (Net MWe): 63
10. Reasons for Restrictions, If Any:
FUEL THERMAL HYDRAULIC LIMITS

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	744.0	147,667.0
12. Hours Reactor Was Critical	422.2	422.2	100,555.2
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	409.5	409.5	98,408.4
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	81,035	81,035	18,198,965
17. Gross Electrical Energy (MWH)	24,716	24,716	5,760,632
18. Net Electrical Energy (MWH)	23,268	23,268	5,451,605
19. Unit Service Factor	55.0	55.0	66.6
20. Unit Availability Factor	55.0	55.0	66.6
*21. Unit Capacity Factor (MDC Net)	49.6	49.6	54.5
22. Unit Capacity Factor (DER Net)	43.4	43.4	51.3
23. Unit Forced Outage Rate	8.7	8.7	25.7

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
1	12/31/79	S	295.7	D	4		IB	INSTRU	REGULATORY SHUTDOWN TO IMPLEMENT REQUIREMENTS OF NUREG-0578.
A	01/13/80	F	3.7	A	3		IB	INSTRU	A. FAILURE OF INTERMEDIATE POWER RANGE MONITOR - SHUTDOWN.
B	01/13/80	F	4.5	A	3		IB	INSTRU	B. FAILURE OF INTERMEDIATE POWER RANGE MONITOR - SHUTDOWN.
C	01/13/80	F	15.2	H	3		IA	INSTRU	C. INTERMEDIATE POWER RANGE MONITOR TRIP ON PERIOD DUE TO PROMPT EFFECT - SHUTDOWN.
2	01/15/80	F	15.4	A	3		CC	INSTRU	2. INTERMEDIATE PRESSURE REGULATOR FAILURE SHUTDOWN.
3	01/18/80	F	.0	A	5				3. INTERMEDIATE PRESSURE REGULATOR TEST - POWER REDUCTION.

SUMMARY

THE UNIT REMAINED OFF-LINE UNTIL 01/12/80 PER REQUIREMENTS OF NUREG-0578. ON 01/13/80, THE UNIT WAS SHUTDOWN DUE TO FAILURE OF THE INTERMEDIATE POWER RANGE MONITOR. AN OUTAGE ALSO OCCURRED ON 01/15/80 FROM INTERMEDIATE PRESSURE REGULATOR FAILURE, WITH A LOAD REDUCTION ON 01/18/80 FOR TESTING. MAXIMUM DEPENDABLE CAPACITY (NET MWE) HAS BEEN DECREASED FROM 65 TO 63 DUE TO FUEL THERMAL HYDRAULIC LIMIT MARGINS.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

BROWNS FERRY 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Alabama/Limestone 10 miles NW of Decatur
2.	Electric Reliability Council	SERC
3.	Type of Reactor	BWR
4.	Power Level (MWT)	3293
5.	Design Electrical Rating (MWE Net)	1065
6.	Date of Initial Criticality	August 17, 1973
7.	Date of Commercial Operation	August 1, 1974
8.	Condenser Cooling Method	Once Through With Forced Draft Cooling Towers
9.	Condenser Cooling Water Source	Tennessee River

Utility & Contractor Information

10.	Licensee	Tennessee Valley Authority
11.	Corporate Address	500A Chestnut Street Tower II Chattanooga, Tennessee 37401
12.	Corporate Contact	H. G. Parris, Manager of Power.
13.	Architect/Engineer	TVA
14.	Nuclear Steam Supply System	GE
15.	Constructor	TVA

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. Lewis
18.	IE Senior Inspector	H. C. Dance
19.	IE Resident Inspector	R. Sullivan
20.	Licensing Project Manager	R. Clark
21.	Docket Number	50-259
22.	License Number/Date of Issue	DPR-33, 6/26/73
23.	Public Document Room	Athens Public Library South and Forrest Athens, Alabama 35611

INSPECTION STATUS

ENFORCEMENT STATUS

AS REQUIRED BY 10 CFR 50, APPENDIX B, CRITERION V, "ACTIVITIES AFFECTING QUALITY SHALL BE PRESCRIBED BY DOCUMENTED INSTRUCTIONS, PROCEDURES, OR DRAWINGS, OF A TYPE APPROPRIATE TO THE CIRCUMSTANCES AND SHALL BE ACCOMPLISHED IN ACCORDANCE WITH THESE INSTRUCTIONS, PROCEDURES OR DRAWINGS". TENNESSEE VALLEY AUTHORITY'S MEMORANDUM DATED OCTOBER 4, 1979 ENTITLED, "PROGRAM TO RESOLVE NRC IE BULLETIN 79-14", STATES IN PART "ALL DISCREPANCIES NOT RESOLVED BY THE EN DES ENGINEER AT THE SITE WILL BE CALLED INTO THE EN DES COORDINATOR (CEB)" AND THE GUIDELINE USED FOR IMPLEMENTATION OF NRC IE BULLETIN 79-14 STATES IN PART "NOTIFY EN DES PROMPTLY REGARDING ALL POSSIBLE NON-CONFORMANCES". CONTRARY TO THE ABOVE, THE FOLLOWING DISCREPANCIES WERE NOT REPORTED TO "EN DES" FOR PROPER EVALUATIONS: 1. THREE HANGERS IN THE RCIC SYSTEM UNIT 3 WERE IDENTIFIED TO HAVE DISCREPANCIES WITH A SIMILAR INSTALLATION IN UNIT 2. THESE DISCREPANCIES WERE NOT REPORTED TO EN DES AND UNIT 2 HANGERS WERE NOT INSPECTED. 2. A HORIZONTAL SUPPORT WAS IDENTIFIED ON A 6" LINE OF RCIC SYSTEM IN UNIT 1 WHICH DID NOT APPEAR ON THE HANGER DRAWING. THIS DISCREPANCY WAS NOT IDENTIFIED TO EN DES AND THE SUPPORT WAS NOT PART OF THE ORIGINAL AS-DESIGNED ANALYSIS.
(7942 INFRACTION)

AS REQUIRED BY 10 CFR 50, APPENDIX B, CRITERION V, "ACTIVITIES AFFECTING QUALITY SHALL BE PRESCRIBED BY DOCUMENTED INSTRUCTIONS, PROCEDURES, OR DRAWINGS, OF A TYPE APPROPRIATE TO THE CIRCUMSTANCES AND SHALL BE ACCOMPLISHED IN ACCORDANCE WITH THESE INSTRUCTIONS, PROCEDURES OR DRAWINGS". TENNESSEE VALLEY AUTHORITY'S MEMORANDUM DATED OCTOBER 4, 1979 ENTITLED, "PROGRAM TO RESOLVE NRC IE BULLETIN 79-14", STATES IN PART "ALL DISCREPANCIES NOT RESOLVED BY THE EN DES ENGINEER AT THE SITE WILL BE CALLED INTO THE EN DES COORDINATOR (CEB)" AND THE GUIDELINE USED FOR IMPLEMENTATION OF NRC IE BULLETIN 79-14 STATES IN PART "NOTIFY EN DES PROMPTLY REGARDING ALL POSSIBLE NON-CONFORMANCES". CONTRARY TO THE ABOVE, THE FOLLOWING DISCREPANCIES WERE NOT REPORTED TO "EN DES" FOR PROPER EVALUATIONS: 1. THREE HANGERS IN THE RCIC SYSTEM UNIT 3 WERE IDENTIFIED TO HAVE DISCREPANCIES WITH A SIMILAR INSTALLATION IN UNIT 2. THESE DISCREPANCIES WERE NOT REPORTED TO EN DES AND UNIT 2 HANGERS WERE NOT INSPECTED. 2. A HORIZONTAL SUPPORT WAS IDENTIFIED ON A 6" LINE OF RCIC SYSTEM IN UNIT 1 WHICH DID NOT APPEAR ON THE HANGER DRAWING. THIS DISCREPANCY WAS NOT IDENTIFIED TO EN DES AND THE SUPPORT WAS NOT PART OF THE ORIGINAL AS-DESIGNED ANALYSIS.
(7942 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

SCHEDULED FORTY DAY REFUELING OUTAGE TO BEGIN JANUARY 4, 1980.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

SHUTDOWN FOR REFUELING ON JANUARY 3, 1980.

◆ LAST IE SITE INSPECTION DATE: JANUARY 4 - FEBRUARY 1, 1980

◆ INSPECTION REPORT NO: 50-259/80-06

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-031/ 03L-0	11/12/79	12/12/79	30 DAY LER	DRYWELL AIR MONITOR RECORDER ERRATIC.
79-033/ 03L-0	11/29/79	12/28/79	30 DAY LER	REACTOR WATER LEVEL INDICATOR SWITCH DRIFT.
79-034/ 03L-0	11/26/79	12/21/79	30 DAY LER	INADVERTENT TRIP OF BOTH RECIRCULATION PUMP MG SETS.
79-035/ 03L-0	12/03/79	01/02/80	30 DAY LER	HIGH PRESSURE COOLANT INJECTION INOPERABLE.
80-01/ 01T-0	01/07/80	01/18/80	14 DAY LER	DIESEL GENERATOR LOSS OF SPEED CONTROL.

SUMMARY

INSPECTION SUMMARY

◆ INSPECTION NOVEMBER 27 - DECEMBER 7 (79-46): THIS ROUTINE INSPECTION INVOLVED 15 RESIDENT INSPECTOR-HOURS IN THE AREAS OF PLANT OPERATIONS, PLANT TOURS, REPORTABLE OCCURRENCES, REFUELING, MAINTENANCE, PLANT PHYSICAL PROTECTION, TRAINING AND RADIATION AREA CONTROLS. OF THE EIGHT AREAS INSPECTED NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION DECEMBER 10-14 (79-45): THIS SPECIAL, UNANNOUNCED INSPECTION INVOLVED 5 INSPECTOR-HOURS ON SITE IN THE AREAS OF UNIT 3 CONTAINMENT INTEGRITY FOLLOWING THE DECEMBER 6, 1979, STARTUP, PLANT TOURS AND PLANT OPERATIONS REVIEW COMMITTEE (PORC) MEETING. OF THE THREE AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION NOVEMBER 26-27 (79-42): THIS SPECIAL, UNANNOUNCED INSPECTION INVOLVED 5 INSPECTOR-HOURS ON SITE IN THE AREA OF SEISMIC ANALYSIS FOR AS-BUILT SAFETY-RELATED PIPING SYSTEMS (IE BULLETIN 79-14). OF THE ONE AREA INSPECTED, ONE ITEM OF NONCOMPLIANCE WAS FOUND IN ONE AREA (INFRACTION - FAILURE TO TAKE CORRECTIVE ACTION).

INSPECTION OCTOBER 15 - NOVEMBER 9 (79-39): THIS ROUTINE INSPECTION INVOLVED 30 RESIDENT INSPECTOR-HOURS IN THE AREAS OF PLANT OPERATIONS, PLANT TOURS, REPORTABLE OCCURRENCES, REFUELING, MAINTENANCE, PLANT PHYSICAL PROTECTION AND RADIATION AREA CONTROLS. OF THE SIX AREAS INSPECTED NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

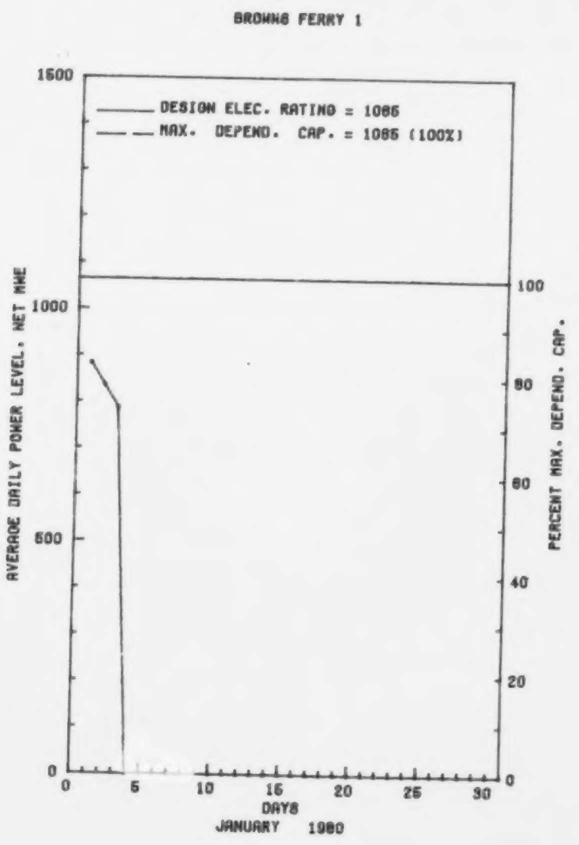
INSPECTION NOVEMBER 19-23 (79-38): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 5 INSPECTOR-HOURS ON SITE IN THE AREA OF FOLLOWUP ON PREVIOUSLY IDENTIFIED OPEN AND UNRESOLVED ITEMS. OF THE ONE AREA INSPECTED, NO ITEM OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 3-19-80

REVIEWED BY NRR DOR: Dick Clark **DATE:** 03-21-80

BROWNS FERRY 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-259
2. Utility Contact: Don Green (205) 729-6846
3. Licensed Thermal Power (MWt): 3293
4. Nameplate Rating (Gross MWe): 1280 x 0.9 = 1152
5. Design Electrical Rating (Net MWe): 1065
6. Maximum Dependable Capacity (Gross MWe): 1098
7. Maximum Dependable Capacity (Net MWe): 1065
8. If Changes Occur In Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	744.0	48,242.0
12. Hours Reactor Was Critical	71.7	71.7	25,349.0
13. Reactor Reserve Shutdown Hours	.0	.0	4,689.5
14. Hours Generator On-Line	71.7	71.7	27,684.6
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	184,152	184,152	74,320,645
17. Gross Electrical Energy (MWH)	61,960	61,960	24,500,420
18. Net Electrical Energy (MWH)	57,164	57,164	23,859,132
19. Unit Service Factor	9.6	9.6	57.4
20. Unit Availability Factor	9.6	9.6	57.4
21. Unit Capacity Factor (MDC Net)	7.2	7.2	46.4
22. Unit Capacity Factor (DER Net)	7.2	7.2	46.4
23. Unit Forced Outage Rate	.0	.0	32.9

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 02/25/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
108	01/03/80	S	674.3	C	2		RC	FUELXX	REFUELING

SUMMARY

THE UNIT WAS SHUTDOWN ON 01/03/80 FOR REFUELING.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

BROWNS FERRY 2

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Alabama/Limestone 10 miles NW of Decatur
2.	Electric Reliability Council	SERC
3.	Type of Reactor	BWR
4.	Power Level (MWT)	3293
5.	Design Electrical Rating (MWE Net)	1065
6.	Date of Initial Criticality	July 20, 1974
7.	Date of Commercial Operation	March 1, 1975
8.	Condenser Cooling Method	Once Through With Forced Draft Cooling Towers
9.	Condenser Cooling Water Source	Tennessee River

Utility & Contractor Information

10.	Licensee	Tennessee Valley Authority
11.	Corporate Address	500A Chestnut Street Tower II Chattanooga, Tennessee 37401
12.	Corporate Contact	H. G. Parris, Manager of Power
13.	Architect/Engineer	TVA
14.	Nuclear Steam Supply System	GE
15.	Constructor	TVA

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. Lewis
18.	IE Senior Inspector	H. C. Dance
19.	IE Resident Inspector	R. Sullivan
20.	Licensing Project Manager	R. Clark
21.	Docket Number	50-260
22.	License Number/Date of Issue	DPR-52, 6/28/74
23.	Public Document Room	Athens Public Library South and Forrest Athens, Alabama 35611

INSPECTION STATUS

ENFORCEMENT STATUS

AS REQUIRED BY 10 CFR 50, APPENDIX B, CRITERION V, "ACTIVITIES AFFECTING QUALITY SHALL BE PRESCRIBED BY DOCUMENTED INSTRUCTIONS, PROCEDURES, OR DRAWINGS, OF A TYPE APPROPRIATE TO THE CIRCUMSTANCES AND SHALL BE ACCOMPLISHED IN ACCORDANCE WITH THESE INSTRUCTIONS, PROCEDURES OR DRAWINGS". TENNESSEE VALLEY AUTHORITY'S MEMORANDUM DATED OCTOBER 4, 1979 ENTITLED, "PROGRAM TO RESOLVE NRC IE BULLETIN 79-14", STATES IN PART "ALL DISCREPANCIES NOT RESOLVED BY THE EN DES ENGINEER AT THE SITE WILL BE CALLED INTO THE EN DES COORDINATOR (CEB)" AND THE GUIDELINE USED FOR IMPLEMENTATION OF NRC IE BULLETIN 79-14 STATES IN PART "NOTIFY EN DES PROMPTLY REGARDING ALL POSSIBLE NON-CONFORMANCES". CONTRARY TO THE ABOVE, THE FOLLOWING DISCREPANCIES WERE NOT REPORTED TO "EN DES" FOR PROPER EVALUATIONS: 1. THREE HANGERS IN THE RCIC SYSTEM UNIT 3 WERE IDENTIFIED TO HAVE DISCREPANCIES WITH A SIMILAR INSTALLATION IN UNIT 2. THESE DISCREPANCIES WERE NOT REPORTED TO EN DES AND UNIT 2 HANGERS WERE NOT INSPECTED. 2. A HORIZONTAL SUPPORT WAS IDENTIFIED ON A 6" LINE OF RCIC SYSTEM IN UNIT 1 WHICH DID NOT APPEAR ON THE HANGER DRAWING. THIS DISCREPANCY WAS NOT IDENTIFIED TO EN DES AND THE SUPPORT WAS NOT PART OF THE ORIGINAL AS-DESIGNED ANALYSIS. (7942 INFRACTION)

AS REQUIRED BY 10 CFR 50, APPENDIX B, CRITERION V, "ACTIVITIES AFFECTING QUALITY SHALL BE PRESCRIBED BY DOCUMENTED INSTRUCTIONS, PROCEDURES, OR DRAWINGS, OF A TYPE APPROPRIATE TO THE CIRCUMSTANCES AND SHALL BE ACCOMPLISHED IN ACCORDANCE WITH THESE INSTRUCTIONS, PROCEDURES OR DRAWINGS". TENNESSEE VALLEY AUTHORITY'S MEMORANDUM DATED OCTOBER 4, 1979 ENTITLED, "PROGRAM TO RESOLVE NRC IE BULLETIN 79-14", STATES IN PART "ALL DISCREPANCIES NOT RESOLVED BY THE EN DES ENGINEER AT THE SITE WILL BE CALLED INTO THE EN DES COORDINATOR (CEB)" AND THE GUIDELINE USED FOR IMPLEMENTATION OF NRC IE BULLETIN 79-14 STATES IN PART "NOTIFY EN DES PROMPTLY REGARDING ALL POSSIBLE NON-CONFORMANCES". CONTRARY TO THE ABOVE, THE FOLLOWING DISCREPANCIES WERE NOT REPORTED TO "EN DES" FOR PROPER EVALUATIONS: 1. THREE HANGERS IN THE RCIC SYSTEM UNIT 3 WERE IDENTIFIED TO HAVE DISCREPANCIES WITH A SIMILAR INSTALLATION IN UNIT 2. THESE DISCREPANCIES WERE NOT REPORTED TO EN DES AND UNIT 2 HANGERS WERE NOT INSPECTED. 2. A HORIZONTAL SUPPORT WAS IDENTIFIED ON A 6" LINE OF RCIC SYSTEM IN UNIT 1 WHICH DID NOT APPEAR ON THE HANGER DRAWING. THIS DISCREPANCY WAS NOT IDENTIFIED TO EN DES AND THE SUPPORT WAS NOT PART OF THE ORIGINAL AS-DESIGNED ANALYSIS. (7942 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

ELECTRO-HYDRAULIC CONTROL SYSTEM PRESSURE REGULATOR MALFUNCTION. TROUBLESHOOTING IN PROGRESS.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

SEVENTY PERCENT OF FULL POWER OPERATION.

LAST IE SITE INSPECTION DATE: JANUARY 4 - FEBRUARY 1, 1980

INSPECTION REPORT NO: 50-260/80-05

SUMMARY

INSPECTION SUMMARY

INSPECTION NOVEMBER 27 - DECEMBER 7 (79-46): THIS ROUTINE INSPECTION INVOLVED 15 RESIDENT INSPECTOR-HOURS IN THE AREAS OF PLANT OPERATIONS, PLANT TOURS, REPORTABLE OCCURRENCES, REFUELING, MAINTENANCE, PLANT PHYSICAL PROTECTION, TRAINING AND RADIATION AREA CONTROLS. OF THE EIGHT AREAS INSPECTED NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION DECEMBER 10-14 (79-45): THIS SPECIAL, UNANNOUNCED INSPECTION INVOLVED 5 INSPECTOR-HOURS ON SITE IN THE AREAS OF UNIT 3 CONTAINMENT INTEGRITY FOLLOWING THE DECEMBER 6, 1979, STARTUP, PLANT TOURS AND PLANT OPERATIONS REVIEW COMMITTEE (PORC) MEETING. OF THE THREE AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-024/ 03L-0	11/21/79	12/19/79	30 DAY LER	MAIN STEAM PRESSURE SWITCH DRIFT.
79-025/ 03L-0	12/05/79	01/04/80	30 DAY LER	MAIN STEAM HIGH FLOW SWITCH DRIFT.

INSPECTION NOVEMBER 26-27 (79-42): THIS SPECIAL, UNANNOUNCED INSPECTION INVOLVED 4 INSPECTOR-HOURS ON SITE IN THE AREA OF SEISMIC ANALYSIS FOR AS-BUILT SAFETY-RELATED PIPING SYSTEMS (IE BULLETIN 79-14). OF THE ONE AREA INSPECTED, ONE ITEM OF NONCOMPLIANCE WAS FOUND IN ONE AREA (INFRACTION - FAILURE TO TAKE CORRECTIVE ACTION).

INSPECTION OCTOBER 15 - NOVEMBER 9 (79-39): THIS ROUTINE INSPECTION INVOLVED 29 RESIDENT INSPECTOR-HOURS IN THE AREAS OF PLANT OPERATIONS, PLANT TOURS, REPORTABLE OCCURRENCES, REFUELING, MAINTENANCE, PLANT PHYSICAL PROTECTION AND RADIATION AREA CONTROLS. OF THE SIX AREAS INSPECTED NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

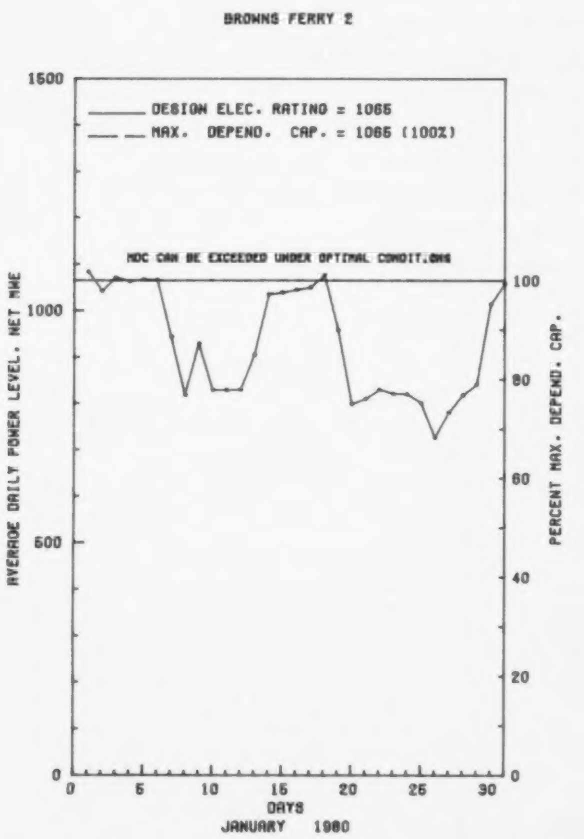
INSPECTION NOVEMBER 19-23 (79-38): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 5 INSPECTOR-HOURS ON SITE IN THE AREA OF FOLLOWUP ON PREVIOUSLY IDENTIFIED OPEN AND UNRESOLVED ITEMS. OF THE ONE AREA INSPECTED, NO ITEM OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 3-19-80

REVIEWED BY NRR DOR: Dick Clark **DATE:** 03-21-80

BROWNS FERRY 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-260
2. Utility Contact: Don Green (205) 729-6846
3. Licensed Thermal Power (MWt): 3293
4. Nameplate Rating (Gross MWe): 1280 x 0.9 = 1152
5. Design Electrical Rating (Net MWe): 1065
6. Maximum Dependable Capacity (Gross MWe): 1098
7. Maximum Dependable Capacity (Net MWe): 1065
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	744.0	43,153.3
12. Hours Reactor Was Critical	744.0	744.0	25,228.4
13. Reactor Reserve Shutdown Hours	.0	.0	11,774.9
14. Hours Generator On-Line	744.0	744.0	24,408.5
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	2,098,111	2,098,111	68,255,111
17. Gross Electrical Energy (MWH)	706,170	706,170	22,656,238
18. Net Electrical Energy (MWH)	688,981	688,981	22,020,450
19. Unit Service Factor	100.0	100.0	56.6
20. Unit Availability Factor	100.0	100.0	56.6
21. Unit Capacity Factor (MDC Net)	87.0	87.0	47.9
22. Unit Capacity Factor (DER Net)	87.0	87.0	47.9
23. Unit Forced Outage Rate	.0	.0	36.1

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type¹	Duration (Hours)	Reason²	Method of Shutting Down Reactor³	Licensee Event Report No.	System Code⁴	Component Code⁵	Cause & Corrective Action to Prevent Recurrence
113	01/07/80	F	.0	A	5				DERATED FOR MAINTENANCE TO "A" CONDENSATE PUMP (HIGH VIBRATION).
114	01/19/80	F	.0	A	5				DERATED FOR "C" REACTOR FEEDWATER PUMP MAINTENANCE (HIGH VIBRATION).
115	01/21/80	S	.0	B	5				DERATED FOR "B" RECIRCULATION PUMP MG SET BRUSH REPLACEMENT.
116	01/25/80	S	.0	H	5				DERATED FOR CONTROL ROD PATTERN ADJUSTMENT.

SUMMARY

OPERATION WAS ROUTINE DURING THE REPORT PERIOD, WITH LOAD REDUCTIONS FOR CONTROL ROD ADJUSTMENT AND FOR MAINTENANCE ON THE "A" CONDENSATE PUMP, THE "C" REACTOR FEEDWATER PUMP, AND THE "B" RECIRCULATION PUMP MOTOR GENERATOR SET.

LEGEND

¹Type:
- F--Forced
- S--Scheduled

²Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

³Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

⁴Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

⁵Exhibit H: Same source.

BROWNS FERRY 3

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Alabama/Limestone 10 miles NW of Decatur
2.	Electric Reliability Council	SERC
3.	Type of Reactor	BWR
4.	Power Level (MWT)	3293
5.	Design Electrical Rating (MWE Net)	1065
6.	Date of Initial Criticality	August 8, 1976
7.	Date of Commercial Operation	March 1, 1977
8.	Condenser Cooling Method	Once Through With Forced Draft Cooling Tower
9.	Condenser Cooling Water Source	Tennessee River

Utility & Contractor Information

10.	Licensee	Tennessee Valley Authority
11.	Corporate Address	500A Chestnut Street Tower II Chattanooga, Tennessee 37401
12.	Corporate Contact	H. G. Parris, Manager of Power
13.	Architect/Engineer	TVA
14.	Nuclear Steam Supply System	GE
15.	Constructor	TVA

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R.C. Lewis
18.	IE Senior Inspector	H. C. Dance
19.	IE Resident Inspector	R. Sullivan
20.	Licensing Project Manager	R. Clark
21.	Docket Number	50-296
22.	License Number/Date of Issue	DPR-68, 7/2/76
23.	Public Document Room	Athens Public Library South and Forrest Athens, Alabama 35611

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO 10 CFR 50.54(O) AND SECTION II.E. AND II.F. OF APPENDIX J TO 10 CFR 50 THE LICENSEE REMOVED THE NORMAL HATCH COVER AND INSTALLED A TEMPORARY HATCH COVER ON THE CONTROL ROD DRIVE HATCH PENETRATION DURING THE TYPE A CONTAINMENT INTEGRATED LEAK RATE TEST PERFORMED NOVEMBER 23-25, 1979. THIS IS AN INFRACTION.
(7938 INFRACTION)

AS REQUIRED BY 10 CFR 50, APPENDIX B, CRITERION V, "ACTIVITIES AFFECTING QUALITY SHALL BE PRESCRIBED BY DOCUMENTED INSTRUCTIONS, PROCEDURES, OR DRAWINGS, OF A TYPE APPROPRIATE TO THE CIRCUMSTANCES AND SHALL BE ACCOMPLISHED IN ACCORDANCE WITH THESE INSTRUCTIONS, PROCEDURES OR DRAWINGS". TENNESSEE VALLEY AUTHORITY'S MEMORANDUM DATED OCTOBER 4, 1979 ENTITLED, "PROGRAM TO RESOLVE NRC IE BULLETIN 79-14", STATES IN PART "ALL DISCREPANCIES NOT RESOLVED BY THE EN DES ENGINEER AT THE SITE WILL BE CALLED INTO THE EN DES COORDINATOR (CEB)" AND THE GUIDELINE USED FOR IMPLEMENTATION OF NRC IE BULLETIN 79-14 STATES IN PART "NOTIFY EN DES PROMPTLY REGARDING ALL POSSIBLE NON-CONFORMANCES". CONTRARY TO THE ABOVE, THE FOLLOWING DISCREPANCIES WERE NOT REPORTED TO "EN DES" FOR PROPER EVALUATIONS: 1. THREE HANGERS IN THE RCIC SYSTEM UNIT 3 WERE IDENTIFIED TO HAVE DISCREPANCIES WITH A SIMILAR INSTALLATION IN UNIT 2. THESE DISCREPANCIES WERE NOT REPORTED TO EN DES AND UNIT 2 HANGERS WERE NOT INSPECTED. 2. A HORIZONTAL SUPPORT WAS IDENTIFIED ON A 6" LINE OF RCIC SYSTEM IN UNIT 1 WHICH DID NOT APPEAR ON THE HANGER DRAWING. THIS DISCREPANCY WAS NOT IDENTIFIED TO EN DES AND THE SUPPORT WAS NOT PART OF THE ORIGINAL AS-DESIGNED ANALYSIS.
(7942 INFRACTION)

AS REQUIRED BY 10 CFR 50, APPENDIX B, CRITERION V, "ACTIVITIES AFFECTING QUALITY SHALL BE PRESCRIBED BY DOCUMENTED INSTRUCTIONS, PROCEDURES, OR DRAWINGS, OF A TYPE APPROPRIATE TO THE CIRCUMSTANCES AND SHALL BE ACCOMPLISHED IN ACCORDANCE WITH THESE INSTRUCTIONS, PROCEDURES OR DRAWINGS". TENNESSEE VALLEY AUTHORITY'S MEMORANDUM DATED OCTOBER 4, 1979 ENTITLED, "PROGRAM TO RESOLVE NRC IE BULLETIN 79-14", STATES IN PART "ALL DISCREPANCIES NOT RESOLVED BY THE EN DES ENGINEER AT THE SITE WILL BE CALLED INTO THE EN DES COORDINATOR (CEB)" AND THE GUIDELINE USED FOR IMPLEMENTATION OF NRC IE BULLETIN 79-14 STATES IN PART "NOTIFY EN DES PROMPTLY REGARDING ALL POSSIBLE NON-CONFORMANCES". CONTRARY TO THE ABOVE, THE FOLLOWING DISCREPANCIES WERE NOT REPORTED TO "EN DES" FOR PROPER EVALUATIONS: 1. THREE HANGERS IN THE RCIC SYSTEM UNIT 3 WERE IDENTIFIED TO HAVE DISCREPANCIES WITH A SIMILAR INSTALLATION IN UNIT 2. THESE DISCREPANCIES WERE NOT REPORTED TO EN DES AND UNIT 2 HANGERS WERE NOT INSPECTED. 2. A HORIZONTAL SUPPORT WAS IDENTIFIED ON A 6" LINE OF RCIC SYSTEM IN UNIT 1 WHICH DID NOT APPEAR ON THE HANGER DRAWING. THIS DISCREPANCY WAS NOT IDENTIFIED TO EN DES AND THE SUPPORT WAS NOT PART OF THE ORIGINAL AS-DESIGNED ANALYSIS.
(7942 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

OPERATING AT ESSENTIALLY 100% POWER AT END OF MONTH.

* LAST IE SITE INSPECTION DATE: JANUARY 4 - FEBRUARY 1, 1980

* INSPECTION REPORT NO: 50-296/80-05

SUMMARY

INSPECTION SUMMARY

* INSPECTION NOVEMBER 27 - DECEMBER 7 (79-46): THIS ROUTINE INSPECTION INVOLVED 15 RESIDENT INSPECTOR-HOURS IN THE AREAS OF PLANT OPERATIONS, PLANT TOURS, REPORTABLE OCCURRENCES, REFUELING, MAINTENANCE, PLANT PHYSICAL PROTECTION, TRAINING AND RADIATION AREA CONTROLS. OF THE EIGHT AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-021/ 03L-0	12/08/79	12/21/79	30 DAY LER	CONTAINMENT DIFFERENTIAL PRESSURE LESS THAN 1.3 PSIG.
79-021/ 01T-0	12/08/79	12/19/79	14 DAY LER	PRIMARY CONTAINMENT INTEGRITY DEGRADED.
79-022/ 03L-0	12/08/79	01/04/80	30 DAY LER	UNABLE TO MAINTAIN PRIMARY CONTAINMENT DIFFERENTIAL PRESSURE - EQUIPMENT HATCH LEAKING.
79-023/ 03L-0	12/02/79	12/31/79	30 DAY LER	DIESEL GENERATOR TRIPPED ON TEST.
79-024/ 03L-0	12/08/79	01/07/80	30 DAY LER	DRYWELL EQUIPMENT HATCH EXCESSIVE LOCAL LEAKAGE.
79-025/ 03L-0	12/12/79	01/10/80	30 DAY LER	REACTOR CORE ISOLATION COOLING FLOW LOW.
79-026/ 03L-0	12/07/79	01/04/80	30 DAY LER	REACTOR CORE ISOLATION COOLING FAILED TO PRODUCE RATED FLOW.
79-027/ 03L-0	12/16/79	01/15/80	30 DAY LER	AVERAGE POWER RANGE MONITOR SETPOINT DRIFT.
79-028/ 03L-0	12/18/79	01/15/80	30 DAY LER	REACTOR PRESSURE SWITCH SETPOINT DRIFT.

INSPECTION DECEMBER 10-14 (79-45): THIS SPECIAL, UNANNOUNCED INSPECTION INVOLVED 68 INSPECTOR-HOURS ON SITE IN THE AREAS OF UNIT 3 CONTAINMENT INTEGRITY FOLLOWING THE DECEMBER 6, 1979, STARTUP, PLANT TOURS AND PLANT OPERATIONS REVIEW COMMITTEE (PORC) MEETING. OF THE THREE AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN TWO AREAS; THREE APPARENT ITEMS OF NONCOMPLIANCE WERE FOUND IN ONE AREA (VIOLATION - FAILURE TO CONTROL ACTIVITIES THAT INVOLVE SAFETY AS EVIDENCED BY LOSS OF PRIMARY CONTAINMENT INTEGRITY; VIOLATION - FAILURE TO SHUT DOWN THE REACTOR AFTER NOT MEETING A LIMITING CONDITION FOR OPERATION (LCO); INFRACTION - FAILURE TO MAKE TIMELY NOTIFICATION TO THE NRC OF AN EVENT REQUIRING PROMPT NOTIFICATION).

INSPECTION NOVEMBER 26-27 (79-42): THIS SPECIAL, UNANNOUNCED INSPECTION INVOLVED 4 INSPECTOR-HOURS ON SITE IN THE AREA OF SEISMIC ANALYSIS FOR AS-BUILT SAFETY-RELATED PIPING SYSTEMS (IE BULLETIN 79-14). OF THE ONE AREA INSPECTED, ONE ITEM OF NONCOMPLIANCE WAS FOUND IN ONE AREA (INFRACTION - FAILURE TO TAKE CORRECTIVE ACTION).

INSPECTION OCTOBER 15 - NOVEMBER 9 (79-39): THIS ROUTINE INSPECTION INVOLVED 29 RESIDENT INSPECTOR-HOURS IN THE AREAS OF PLANT OPERATIONS, PLANT TOURS, REPORTABLE OCCURRENCES, REFUELING, MAINTENANCE, PLANT PHYSICAL PROTECTION AND RADIATION AREA CONTROLS. OF THE SIX AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION NOVEMBER 19-23 (79-38): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 72 INSPECTOR-HOURS ON SITE IN THE AREAS OF WITNESSING PREPARATIONS AND PERFORMANCE OF THE CONTAINMENT INTEGRATED LEAK RATE TEST AND LOCAL LEAK RATE TESTING, DRYWELL INSPECTION OF HYDRAULIC SHOCK ARRESTORS, FOLLOWUP ON PREVIOUSLY IDENTIFIED OPEN AND UNRESOLVED ITEMS AND GENERAL INSPECTION IN CERTAIN AREAS OF THE REACTOR BUILDING AND DRYWELL. OF THE FOUR AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN THREE AREAS; ONE ITEM OF NONCOMPLIANCE WAS FOUND IN ONE AREA (INFRACTION - THE USE OF A TEMPORARY CRD HATCH COVER DURING THE INTEGRATED LEAK RATE TEST).

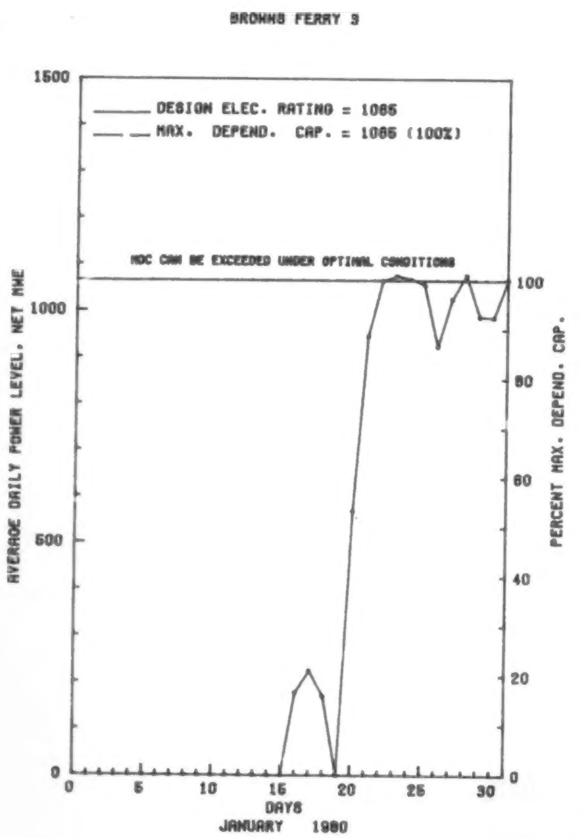
INSPECTION JANUARY 9-11 (80-02): MATERIAL CONTROL AND ACCOUNTING. THE INSPECTION INVOLVED 20 INSPECTOR-HOURS ON SITE BY ONE NRC INSPECTOR AND WAS BEGUN DURING REGULAR HOURS. THE LICENSEE WAS FOUND TO BE IN COMPLIANCE WITH NRC REQUIREMENTS IN THE ONE AREA EXAMINED.

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 3-19-80

REVIEWED BY NRR DOR: Dick Clark **DATE:** 03-21-80

BROWNS FERRY 3

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-296
2. Utility Contact: Don Green (205) 729-6846
3. Licensed Thermal Power (MWt): 3293
4. Nameplate Rating (Gross MWe): 1280 x 0.9 = 1152
5. Design Electrical Rating (Net MWe): 1065
6. Maximum Dependable Capacity (Gross MWe): 1098
7. Maximum Dependable Capacity (Net MWe): 1065
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	744.0	25,608.0
12. Hours Reactor Was Critical	332.4	332.4	19,240.1
13. Reactor Reserve Shutdown Hours	.0	.0	1,658.4
14. Hours Generator On-Line	308.9	308.9	18,746.7
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	912,358	912,358	54,020,224
17. Gross Electrical Energy (MWH)	305,980	305,980	17,708,560
18. Net Electrical Energy (MWH)	294,282	294,282	17,182,955
19. Unit Service Factor	41.5	41.5	73.2
20. Unit Availability Factor	41.5	41.5	73.2
21. Unit Capacity Factor (MDC Net)	37.1	37.1	63.0
22. Unit Capacity Factor (DER Net)	37.1	37.1	63.0
23. Unit Forced Outage Rate	58.5	58.5	10.8

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
112	12/30/79	F	375.3	A	4				INSTALLATION OF OVERHEAD CABLES FROM COOLING TOWERS TRANSFORMERS TO BUS TIE BOARDS CONTINUES.
113	01/17/80	F	14.7	A	3				DURING TESTING NO. 2 STOP VALVE CLOSED CAUSING A SPIKE IN REACTOR PRESSURE.
114	01/18/80	F	45.1	A	3				APRM HIGH FLUX WHILE PERFORMING NO. 2 STOP VALVE TEST. UNIT REMAINED DOWN FOR MAINTENANCE TO THE EHC SYSTEM.
115	01/29/80	F	.0	A	5				DERATED FOR "C" REACTOR FEEDWATER PUMP BALANCING (HIGH VIBRATION).

SUMMARY

THE UNIT WAS SHUTDOWN UNTIL 01/15/80 FOR INSTALLATION OF OVERHEAD CABLES FROM COOLING TOWER TRANSFORMERS TO BUS TIE BOARDS. UNIT SCRAMS OCCURRED ON 01/17/80 AND 01/18/80 DUE TO HIGH-HIGH FLUX WHEN REACTOR PRESSURE SPIKED BECAUSE OF CLOSURE OF THE NO. 2 MAIN STEAM STOP VALVE. ON 01/29/80, LOAD WAS REDUCED FOR "C" REACTOR FEEDWATER PUMP BALANCING.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

BRUNSWICK 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1. Location (state/county) (nearest population center)	North Carolina/Brunswick 3 miles N of Southport
2. Electric Reliability Council	Southeastern
3. Type of Reactor	BWR
4. Power Level (MWT)	2436
5. Design Electrical Rating (MWE Net)	821
6. Date of Initial Criticality	October 8, 1976
7. Date of Commercial Operation	March 18, 1977
8. Condenser Cooling Method	Once Through
9. Condenser Cooling Water Source	Cape Fear River

Utility & Contractor Information

10. Licensee	Carolina Power and Light Company
11. Corporate Address	411 Fayetteville Street Raleigh, North Carolina 27602
12. Corporate Contact	J. A. Jones Executive Vice President
13. Architect/Engineer	United Engineers & Constructors, Inc.
14. Nuclear Steam Supply System	GE
15. Constructor	Brown and Root, Inc.

Regulatory Information

16. IE Region responsible	II
17. IE Branch Chief	R.C. Lewis
18. IE Senior Inspector	P.J. Kellogg
19. IE Resident Inspector	J. E. Ouzts
20. Licensing Project Manager	J. Hannon
21. Docket Number	50-325
22. License Number/Date of Issue	DPR-71, 9/8/76
23. Public Document Room	Southport-Brunswick County Library 109 W. Moore Street Southport, North Carolina 28461

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

+ UNIT IN NORMAL POWER OPERATION.
+ LAST IE SITE INSPECTION DATE: JANUARY 7-11, 1980
+ INSPECTION REPORT NO: 50-325/80-01

SUMMARY

INSPECTION SUMMARY

+ INSPECTION DECEMBER 1-14 (79-44): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 41 INSPECTOR-HOURS ON SITE IN THE AREAS OF PLANT OPERATIONS, PLANT TOURS, RADIATION PROTECTION, QUALITY ASSURANCE, ORGANIZATION AND ADMINISTRATION AND FOLLOWUP OF ABNORMAL EVENTS. THIS ROUTINE AND REACTIVE INSPECTION BY THE RESIDENT INSPECTOR INVOLVED 21 MAN-HOURS OF INSPECTION TIME ON SITE IN THE AREAS IDENTIFIED ABOVE. OF THE SIX AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION JANUARY 7-11 (80-01): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 93 INSPECTOR-HOURS ON SITE AND OFFSITE IN THE AREAS OF COORDINATION WITH OFFSITE SUPPORT AGENCIES; EMERGENCY FACILITIES, EQUIPMENT AND PROCEDURES; MEANS FOR DETERMINING A RELEASE; EMERGENCY TRAINING; EMERGENCY DRILLS; FIRE BRIGADE ORGANIZATION AND TRAINING; EMERGENCY ORGANIZATION; IE BULLETIN 79-18 AND FOLLOWUP ON PREVIOUS INSPECTION FINDINGS. OF THE NINE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
SPECIAL REPORT	11/29/79	12/20/79	SPECIAL REPORT	PRESSURE SWITCH FP-PSL-3963 SHORTED OUT AFFECTING ANNUNCIATOR FOR REACTOR BUILDING ZONE 13, 14, AND 15 SPECIAL REPORT.
79-89 03L-0	11/20/79	12/17/79	30 DAY LER	REACTOR CORE ISOLATION COOLING TURBINE TRIPPED ON MECHANICAL OVERSPEED ON AUTO INITIATION SIGNAL OF MINUS 38 INCHES.
79-91/ 03L-0	12/01/79	12/20/79	30 DAY LER	EXCESSIVE PACKING LEAKAGE ON 1-B RECIRCULATION PUMP SUCTION AND DISCHARGE VALVES.
79-92/ 03L-0	12/04/79	12/20/79	30 DAY LER	CONTAINMENT ATMOSPHERE MONITOR CAC-ATH-1263 FAILED DUE TO BLOWN POWER SUPPLY FUSE.
79-93/ 03L-0	11/27/79	12/19/79	30 DAY LER	HIGH STEAMLINE RADIATION CHANNEL "B" OUT OF CALIBRATION.
79-101/ 03L-0	12/05/79	12/28/79	30 DAY LER	HPCI EXHAUST DIAPHRAM HIGH PRESSURE SWITCH IE41-PS N012A WOULD NOT ACTUATE DURING A CALIBRATION.
79-102/ 03L-0	11/29/79	12/20/79	30 DAY LER	CONTAINMENT ATMOSPHERE MONITOR CAC-AT-1263 TRIPPED PUMP BEARINGS WERE FOUND SIEZED.
79-103/ 03L-0	12/05/79	12/19/79	30 DAY LER	ROOT VALVES TO REACTOR CORE ISOLATION COOLING TURBINE EXHAUST DIAPHRAGM HIGH PRESSURE FOUND CLOSED.
79-104/ 03L-0	12/06/79	12/28/79	30 DAY LER	HPCI SYSTEM ISOLATED DURING TEST DUE TO A HIGH SUCTION PRESSURE SIGNAL CAUSED BY LEAKING DIAPHRAM ON IER1-PSH-N031.
79-105/ 03L-0	12/19/79	01/07/80	30 DAY LER	RIP VALVES X82B AND X85B, F, F NOT OPERATING PROPERLY DURING SURVEILLANCE PRIOR TO STARTUP.
79-106/ 03L-0	12/07/79	12/19/79	30 DAY LER	ROOT VALVES WHICH PROVIDE ANNUNCIATOR FOR CORE SPRAY-RHR PUMP RUNNING WERE FOUND SHUT.
79-107/ 03L-0	12/12/79	12/20/79	30 DAY LER	DAMAGE TO SNUBBERS ON TAILPIPE OF SAFETY RELIEF VALVE F013H.
79-108/ 03L-0	12/11/79	01/03/80	30 DAY LER	DRYWELL PRESSURE INDICATOR CAC-PI-3341 GAVE INCORRECT READING DUE TO ZERO SHIFT.
79-112/ 03L-0	12/19/79	01/07/80	30 DAY LER	DURING NORMAL STARTUP THE 3 INDICATION IN THE TENS DIGIT FOR CONTROL ROD 42-07 DID NOT LIGHT.

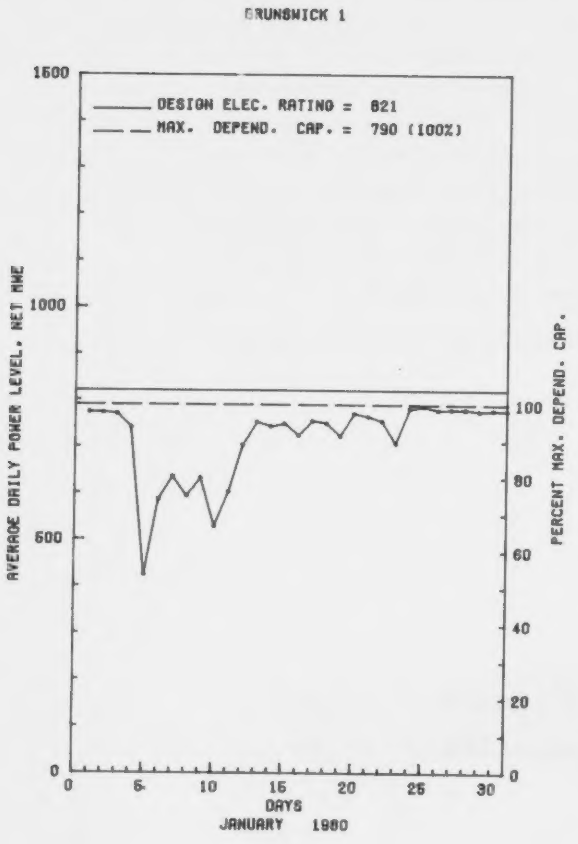
REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 3-19-80

REVIEWED BY NRR DOR: John N. Hannon **DATE:** 3-21-80

best copy available

BRUNSWICK 1

AVERAGE POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-325
2. Utility Contact: S. Willis (919) 457-9521
3. Licensed Thermal Power (MWt): 2436
4. Nameplate Rating (Gross MWe): 963 x 0.9 = 867
5. Design Electrical Rating (Net MWe): 821
6. Maximum Dependable Capacity (Gross MWe): 815
7. Maximum Dependable Capacity (Net MWe): 790
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	744.0	25,201.0
12. Hours Reactor Was Critical	744.0	744.0	18,133.6
13. Reactor Reserve Shutdown Hours	.0	.0	1,647.1
14. Hours Generator On-Line	744.0	744.0	17,076.8
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,646,831	1,646,831	35,476,327
17. Gross Electrical Energy (MWH)	551,232	551,232	11,790,051
18. Net Electrical Energy (MWH)	532,690	532,690	11,340,506
19. Unit Service Factor	100.0	100.0	67.8
20. Unit Availability Factor	100.0	100.0	67.8
21. Unit Capacity Factor (MDC Net)	90.6	90.6	57.0
22. Unit Capacity Factor (DER Net)	87.2	87.2	54.8
23. Unit Forced Outage Rate	.0	.0	20.0

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUELING, 800301, 1680 HOURS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
001	01/04/80	S	7.0	H	5		RC	CONROD	REDUCED POWER AND PERFORMED A ROD IMPROVEMENT SHUFFLE.
002	01/10/80	S	7.0	H	5		RC	CONROD	REDUCED POWER AND PERFORMED A ROD IMPROVEMENT SHUFFLE.

SUMMARY

OPERATION WAS ROUTINE DURING THE REPORT PERIOD, WITH LOAD REDUCTIONS ON 01/04/80 AND 01/10/80 FOR CONTROL ROD IMPROVEMENT SHUFFLES.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

BRUNSWICK 2

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	North Carolina/Brunswick 3 miles N of Southport
2.	Electric Reliability Council	Southeastern
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2436
5.	Design Electrical Rating (MWE Net)	821
6.	Date of Initial Criticality	March 20, 1975
7.	Date of Commercial Operation	November 3, 1975
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Cape Fear River

Utility & Contractor Information

10.	Licensee	Carolina Power and Light Company
11.	Corporate Address	411 Fayetteville Street Raleigh, North Carolina 27602
12.	Corporate Contact	J. A. Jones Executive Vice President
13.	Architect/Engineer	United Engineers & Constructors, Inc.
14.	Nuclear Steam Supply System	GE
15.	Constructor	Brown and Root, Inc.

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R.C. Lewis
18.	IE Senior Inspector	P.J. Kellogg
19.	IE Resident Inspector	J. E. Ouzts
20.	Licensing Project Manager	J. Hannon
21.	Docket Number	50-324
22.	License Number/Date of Issue	DPR-62, 12/27/74
23.	Public Document Room	Southport-Brunswick County Library 109 W. Moore Street Southport, North Carolina 28461

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

UNIT IN NORMAL POWER OPERATION.

♦ LAST IE SITE INSPECTION DATE: JANUARY 7-11, 1980

♦ INSPECTION REPORT NO: 50-324/80-01

SUMMARY

INSPECTION SUMMARY

♦ INSPECTION DECEMBER 1-14 (79-40): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 41 INSPECTOR-HOURS ON SITE IN THE AREAS OF PLANT OPERATIONS, PLANT TOURS, RADIATION PROTECTION, QUALITY ASSURANCE, ORGANIZATION AND ADMINISTRATION AND FOLLOWUP OF ABNORMAL EVENTS. THIS ROUTINE INSPECTION BY THE RESIDENT INSPECTOR INVOLVED 20 MAN-HOURS OF INSPECTION IN THE FIRST FIVE AREAS IDENTIFIED ABOVE. OF THE FIVE AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION JANUARY 7-11 (80-01): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 93 INSPECTOR-HOURS ON SITE AND OFFSITE IN THE AREAS OF COORDINATION WITH OFFSITE SUPPORT AGENCIES; EMERGENCY FACILITIES, EQUIPMENT AND PROCEDURES; MEANS FOR DETERMINING A RELEASE; EMERGENCY TRAINING; EMERGENCY DRILLS; FIRE BRIGADE ORGANIZATION AND TRAINING; EMERGENCY ORGANIZATION; IE BULLETIN 79-18 AND FOLLOWUP ON PREVIOUS INSPECTION FINDINGS. OF THE NINE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

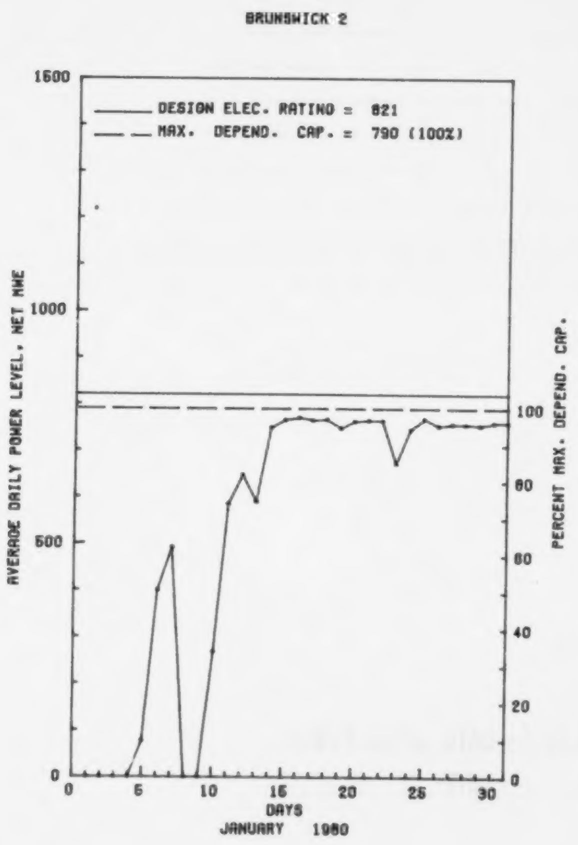
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-88/ 03L-0	12/06/79	12/28/79	30 DAY LER	TRIP OF 2B RECIRCULATION PUMP CAUSED BY BLOWN FUSES IN VOLTAGE REGULATOR CIRCUIT.
79-90/ 03L-0	12/14/79	01/09/80	30 DAY LER	LHGR LIMIT EXCEEDED FOLLOWING A RECIRCULATION PUMP TRIP.
79-91/ 03L-0	11/26/79	12/20/79	30 DAY LER	BLOCKED SAMPLE FLOW-CONTAINMENT MONITOR CAC-AR-1259 DUE TO MOISTURE ENTRAPMENT.
79-93/ 03L-0	11/20/79	12/07/79	30 DAY LER	ROD NO. 42-33 GAVE ROD OVER TRAVEL ALARM WHEN WITHDRAWN TO NOTCH 48.
79-98/ 03L-0	11/29/79	12/21/79	30 DAY LER	ELECTRICAL SHORT ON FLOW SWITCH E11-FSL-N021A DURING MAINTENANCE.
79-101/ 03L-0	11/20/79	12/14/79	30 DAY LER	INTERMEDIATE RANGE MONITOR "D" DID NOT INDICATE PROPER OVERLAP ON RANGE 7.
79-102/ 03L-0	12/14/79	01/03/80	30 DAY LER	VALVE POSITION INDICATION FOR DRYWELL TO TORUS VACUUM BREAKER X18H FAILED.
79-103/ 03L-0	11/26/79	12/19/79	30 DAY LER	CHLORINE TANKER IN EXCLUSION AREA WITH BUILDING CHLORINE DETECTORS OUT OF SERVICE.
79-105/ 03L-0	12/01/79	12/14/79	30 DAY LER	ROD BLOCK MONITOR "B" TRIPPED DUE TO SHORT OF AN INDICATING LIGHT BASE.
79-106/ 03L-0	12/15/79	01/03/80	30 DAY LER	CONTAINMENT ATMOSPHERE MONITORS CAC-AT-1259 FAILED DUE TO LOW FLOW.
79-109/ 01P-0	12/18/79	12/28/79	30 DAY LER	THRU WALL LEAK ON BASE METAL OF 1-INCH TO 3/4-INCH REDUCER ON 2A RECIRCULATION PUMP SUCTION LINE TEST CONNECTION.

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 3-19-80

REVIEWED BY NRR DOR: John N. Hannon **DATE:** 3-21-80

BRUNSWICK 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-324
2. Utility Contact: E. Willis (919) 457-9521
3. Licensed Thermal Power (MWt): 2436
4. Nameplate Rating (Gross MWe): 963 x 0.9 = 867
5. Design Electrical Rating (Net MWe): 821
6. Maximum Dependable Capacity (Gross MWe): 815
7. Maximum Dependable Capacity (Net MWe): 790
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	744.0	37,225.0
12. Hours Reactor Was Critical	703.0	703.0	26,026.9
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	585.8	585.8	24,459.8
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,229,990	1,229,990	46,564,452
17. Gross Electrical Energy (MWH)	411,690	411,690	15,225,764
18. Net Electrical Energy (MWH)	397,659	397,659	14,478,131
19. Unit Service Factor	78.7	78.7	65.7
20. Unit Availability Factor	78.7	78.7	65.7
21. Unit Capacity Factor (MDC Net)	67.7	67.7	49.1
22. Unit Capacity Factor (DER Net)	65.1	65.1	47.2
23. Unit Forced Outage Rate	8.1	8.1	12.0

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUELING, 800426, 1848 HOURS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
001	12/25/79	S	106.4	D	4		SH	VALVEX	THE UNIT WAS SEPARATED FROM THE GRID TO INSTALL A SAFETY RELIEF VALVE POSITION INDICATION SYSTEM ON ALL PRIMARY RELIEF VALVES AND FOR A SCHEDULED PIPE SNUBBER INSPECTION. THE OUTAGE WAS EXTENDED DUE TO: (1) A LEAK WAS DISCOVERED ON A REACTOR RECIRC TEST CONNECTION BUSHING. (2) REACTOR VESSEL CHEMISTRY WENT OUT OF SPECIFICATION WHEN ATTEMPTING TO ESTABLISH A CONDENSER VACUUM.
002	01/07/80	F	51.9	A	1		SH	VALVEX	THE UNIT WAS SHUTDOWN BECAUSE OF HIGH TAIL PIPE TEMPERATURE ON SAFETY RELIEF VALVE F013K WHICH INDICATES VALVE LEAKAGE. THE UNIT #2 F013K SAFETY RELIEF VALVE WAS REMOVED FROM THE SYSTEM; ALSO AS A PRECAUTIONARY MEASURE, THE F013A SAFETY RELIEF VALVE WAS REMOVED.

SUMMARY

THE UNIT WAS SHUTDOWN UNTIL 01/04/80 FOR PLANT MODIFICATION TO THE SAFETY RELIEF VALVES. AN OUTAGE ALSO OCCURRED ON 01/07/80 BECAUSE OF LEAKAGE ON SAFETY RELIEF VALVE F013K. OPERATION WAS ROUTINE FOR THE REMAINDER OF THE REPORT PERIOD.

LEGEND

[1] Type:
F-Forced
S-Scheduled

[2] Reason:
A-Equipment Failure
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error
H-Other

[3] Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Continuations
5-Load Reduction
9-Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

CALVERT CLIFFS 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Maryland/Calvert 40 miles S of Annapolis
2.	Electric Reliability Council	MAAC
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2700
5.	Design Electrical Rating (MWE Net)	845
6.	Date of Initial Criticality	October 7, 1974
7.	Date of Commercial Operation	May 8, 1975
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Chesapeake Bay

Utility & Contractor Information

10.	Licensee	Baltimore Gas and Electric Company
11.	Corporate Address	Gas and Electric Building Baltimore, Maryland 21203
12.	Corporate Contact	A. E. Lundvall, Jr. Vice President-Supply
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Combustion Engineering
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Senior Inspector	R. Keimig
19.	IE Principal Inspector	D. F. Johnson
20.	Licensing Project Manager	E. L. Conner, Jr.
21.	Docket Number	50-317
22.	License Number/Date of Issue	DPR-53, 7/31/74
23.	Public Document Room	Calvert County Library Prince Frederick, Maryland 20678

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

+ None

FACILITY ITEMS (PLANS AND PROCEDURES):

None

MANAGERIAL ITEMS:

+ None

PLANT STATUS:

+ Unit was restarted on 2/11 following a two week outage to install complete modifications required by TMI-2 Short Term Lessons learned.

+ LAST IE SITE INSPECTION DATE: 2/11/80

+ INSPECTION REPORT NO: 50-317/80-01

SUMMARY

INSPECTION SUMMARY

+ No reports issued during this period.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

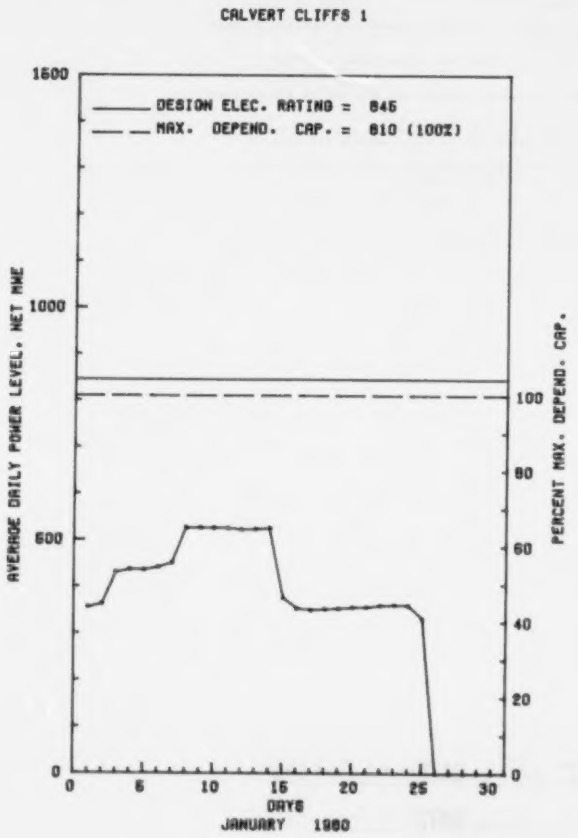
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-062/03L	10/19/79	11/06/79	30 Day LER	Channel A high power trip was placed in bypass due to spurious trips
79-067/03L	11/26/79	12/20/79	30 Day LER	Channel A reactor protective trips bypassed due to spurious signals from cold leg rtd input
79-073/03L	12/04/79	01/04/80	30 Day LER	Diesel gen 11 OOS to repair binding output breaker auxiliary switch
79-074/03L	12/04/79	12/20/79	30 Day LER	No. 12 emergency diesel generator removed from service to repair fuel oil leaks
79-076/03L	12/10/79	01/08/80	30 Day LER	Salt water loop 11 removed from service for maintenance
79-077/03L	12/13/79	01/10/80	30 Day LER	Low oil pressure tripped no. 12 control room air conditioning compressor
79-079/03L	12/19/79	01/10/80	30 Day LER	Both spent fuel pool exhaust fans were taken out of service (OOS) to replace roughing filters
79-080/04T	12/18/79	01/02/80	14 Day LER	Oyster samples at Camp Conoy had Ag-110m concentration more than 30X background, with a new technique reducing the minimum detectable level and no releases above limits

REVIEWED BY OMPA LOEB: Mk Beebe **DATE:** 03/19/80

REVIEWED BY NRR DOR: E L Conner, Jr. **DATE:** 3/20/80

CALVERT CLIFFS 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-317
2. Utility Contact: S. D. Merson (301) 234-5240
3. Licensed Thermal Power (MWt): 2700
4. Nameplate Rating (Gross MWe): 1020 x 0.9 = 918
5. Design Electrical Rating (Net MWe): 845
6. Maximum Dependable Capacity (Gross MWe): 845
7. Maximum Dependable Capacity (Net MWe): 810
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): VARIOUS
10. Reasons for Restrictions, If Any:
UNEQUAL POWER DISTRIBUTION OF CORE & TURBINE BLADE PROBLEMS.

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	744.0	41,509.0
12. Hours Reactor Was Critical	600.4	600.4	33,940.0
13. Reactor Reserve Shutdown hours	.0	.0	1,060.5
14. Hours Generator On-Line	600.0	600.0	32,302.0
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	929,727	929,727	76,266,715
17. Gross Electrical Energy (MWH)	272,128	272,128	25,034,658
18. Net Electrical Energy (MWH)	250,462	250,462	23,899,668
19. Unit Service Factor	80.6	80.6	77.8
20. Unit Availability Factor	80.6	80.6	77.8
21. Unit Capacity Factor (MDC Net)	41.6	41.6	71.1
22. Unit Capacity Factor (DER Net)	39.8	39.8	68.1
23. Unit Forced Outage Rate	.0	.0	8.9

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 03/03/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
79-15	11/08/79	F	600.0	H	5		RC	ZZZZZZ	FORCED REDUCTIONS TO VARIOUS LOADS DUE TO UNEQUAL POWER DISTRIBUTION OF THE CORE.
80-01	01/25/80	S	144.0	D	9		ZZ	ZZZZZZ	TO PERFORM TMI MODIFICATIONS AS REQUIRED BY NRC.

SUMMARY

OPERATION WAS UNDER VARIOUS REDUCED LOADS DUE TO UNEQUAL POWER DISTRIBUTION IN THE CORE AND TURBINE BLADE PROBLEMS. THE UNIT WAS SHUTDOWN ON 01/25/80 FOR INSTALLATION OF NRC-REQUIRED PLANT MODIFICATIONS.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

CALVERT CLIFFS 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Maryland/Calvert 40 miles S of Annapolis
2.	Electric Reliability Council	MAAC
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2700
5.	Design Electrical Rating (MWE Net)	845
6.	Date of Initial Criticality	November 30, 1976
7.	Date of Commercial Operation	April 1, 1977
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Chesapeake Bay

Utility & Contractor Information

10.	Licensee	Baltimore Gas and Electric Company
11.	Corporate Address	Gas and Electric Building Baltimore, Maryland 21203
12.	Corporate Contact	A. E. Lundvall, Jr. Vice President-Supply
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Combustion Engineering
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	IE Senior Inspector	R. Keimig
19.	IE Principal Inspector	D. F. Johnson
20.	Licensing Project Manager	E. L. Conner, Jr.
21.	Docket Number	50-318
22.	License Number/Date of Issue	DPR-69, 8/13/76
23.	Public Document Room	Calvert County Library Prince Frederick, Maryland 20678

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

```
      I N S P E C T I O N   S T A T U S

      OTHER ITEMS OF INTEREST
      SYSTEMS AND COMPONENT PROBLEMS:
      None
      FACILITY ITEMS (PLANS AND PROCEDURES):
      None.
      MANAGERIAL ITEMS:
 +    None
      PLANT STATUS:
 +    No significant information to report.
 + LAST IE SITE INSPECTION DATE:  2/11/80
 +          INSPECTION REPORT NO:  50-318/80-01
      SUMMARY
      INSPECTION SUMMARY
 +    No reports issued during this period.
```

REPORTS RECEIVED FROM LICENSEE

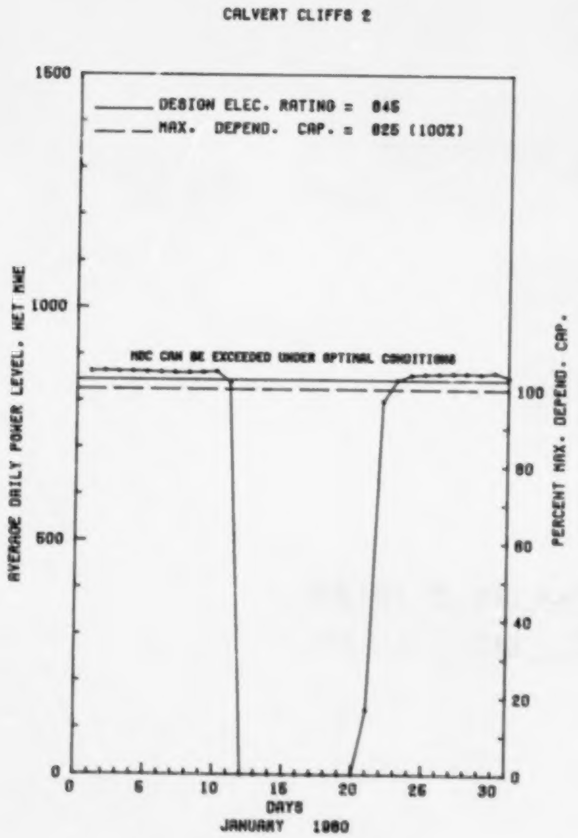
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-042/ 03L	11/27/79	12/20/79	30 Day LER	Pressurizer pressure transmitter for ESF zero shifted out of calibration due to physical contact of transmitter with a fire system hanger
79-043/ 03L	12/12/79	01/03/80	30 Day LER	"A" wide range nuclear instrument inoperable (caused by open electrical connection)
79-044/ 03L	12/12/79	01/10/80	30 Day LER	Channel B axial flux offset was out of specification
79-045/ 03L	12/06/79	01/03/80	30 Day LER	"B" steam generator level reading 15" low due to transmitter fault
79-046/ 03L	12/14/79	01/10/80	30 Day LER	During steady state operation the incore detector system was inoperable during plant computer corrective maintenance
79-047 03L	12/0?/79	01/04/80	30 Day LER	Reactor manually tripped during low power physics testing when TAV dropped below 515 degrees F
79-048/ 03L	12/17/79	01/04/80	30 Day LER	Iodine filter 22 oos due to impeded motion of ckt bkr contact pivot arm
79-049/ 03L	12/21/79	01/21/80	30 Day LER	"B" cold leg temp oos due to erratic indication caused by broken lead on current loop resistor

REVIEWED BY OMPA LOEB: DATE: 03/19/80

REVIEWED BY NRR DOR: DATE: 3/30/80

CALVERT CLIFFS 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-318
2. Utility Contact: S. D. Merson (301) 234-5240
3. Licensed Thermal Power (MWt): 2700
4. Nameplate Rating (Gross MWe): 1012 x 0.9 = 911
5. Design Electrical Rating (Net MWe): 845
6. Maximum Dependable Capacity (Gross MWe): 860
7. Maximum Dependable Capacity (Net MWe): 825
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level to Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	744.0	24,865.0
12. Hours Reactor Was Critical	524.9	524.9	20,569.8
13. Reactor Reserve Shutdown Hours	.0	.0	413.1
14. Hours Generator On-Line	516.7	516.7	20,125.2
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,357,176	1,357,176	49,671,467
17. Gross Electrical Energy (MWH)	454,186	454,186	16,439,599
18. Net Electrical Energy (MWH)	433,092	433,092	15,490,112
19. Unit Service Factor	69.4	69.4	80.9
20. Unit Availability Factor	69.4	69.4	80.9
21. Unit Capacity Factor (MDC Net)	70.6	70.6	76.5
22. Unit Capacity Factor (DER Net)	68.9	68.9	74.7
23. Unit Forced Outage Rate	.0	.0	6.8

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-01	01/12/80	S	227.3	D	1		ZZ	ZZZZZZ	TO PERFORM TMI MODIFICATION AS REQUIRED BY NRC.

SUMMARY

OPERATION WAS AT NEAR FULL POWER DURING THE REPORT PERIOD, WITH AN OUTAGE FROM 01/12/80 THROUGH 01/29/80 FOR INSTALLATION OF NRC-REQUIRED PLANT MODIFICATIONS.

LEGEND

[1] Type:
F–Forced
S–Scheduled

[2] Reason:
A–Equipment Failure
B–Maintenance or Test
C–Refueling
D–Regulatory Restriction
E–Operator Training & License Examination
F–Administrative
G–Operational Error
H–Other

[3] Method:
1–Manual
2–Manual Scram
3–Automatic Scram
4–Continuations
5–Load Reduction
9–Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

COOK 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Michigan/Berrien 11 miles S of Benton Harbor
2.	Electric Reliability Council	ECAR
3.	Type of Reactor	PWR
4.	Power Level (MWT)	3250
5.	Design Electrical Rating (MWE Net)	1054
6.	Date of Initial Criticality	January 18, 1975
7.	Date of Commercial Operation	August 27, 1975
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10.	Licensee	AEP Corporation
11.	Corporate Address	Indiana & Michigan Power Co. 2 Broadway New York, New York 10004
12.	Corporate Contact	John Dolan Senior Executive V. P. Eng.
13.	Architect/Engineer	AEP
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Indiana & Michigan Power Co.

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Senior Inspector	D.C. Boyd
19.	IE Resident Inspector	R. Masse (616) 465-6775
20.	Licensing Project Manager	D. Wigginton
21.	Docket Number	50-315
22.	License Number/Date of Issue	DPR-58 issued 10/25/74
23. *	Public Document Room	Maude Preston Palenske Memorial Library 500 Market Street St. Joseph, Michigan 49085

INSPECTION STATUS

ENFORCEMENT STATUS

10 CFR 71.51(A) REQUIRES THAT A LICENSEE "...SHALL ESTABLISH, MAINTAIN AND EXECUTE A QUALITY ASSURANCE PROGRAM SATISFYING EACH OF THE APPLICABLE CRITERIA SPECIFIED IN APPENDIX E, 'QUALITY ASSURANCE CRITERIA FOR SHIPPING PACKAGES FOR RADIOACTIVE MATERIAL'..." IT FURTHER REQUIRES THAT "THE DESCRIPTION OF THE QUALITY ASSURANCE PROGRAM SHALL INCLUDE A DISCUSSION OF WHICH REQUIREMENTS OF APPENDIX E ARE APPLICABLE AND HOW THEY WILL BE SATISFIED," AND THAT "A DESCRIPTION OF THAT PROGRAM SHALL BE FILED, IN ACCORDANCE WITH THIS SECTION, BY JANUARY 1, 1979, WITH THE DIRECTOR, OFFICE OF NUCLEAR MATERIAL SAFETY AND SAFEGUARDS, NUCLEAR REGULATORY COMMISSION, WASHINGTON, D.C. 20555." CONTRARY TO THE ABOVE, NO QUALITY ASSURANCE PROGRAM FOR SHIPPING PACKAGES WAS IN EFFECT AND NO DESCRIPTION OF SUCH A PROGRAM HAD BEEN FILED WITH THE DIRECTOR, NMSS.
(7926 INFRACTION)

PLANT MANAGER INSTRUCTION PMI 4060 REQUIRES THE LICENSEE TO FORWARD ONLY PROPERLY PREPARED, REVIEWED AND APPROVED TECHNICAL DATA FOR ENTRY IN TECHNICAL DATA BOOKS. CONTRARY TO THE ABOVE, THE DIFFERENTIAL BORON WORTH CURVES IN THE TECHNICAL DATA BOOKS HAVE NOT BEEN SCALED UP TO REFLECT THE DISCREPANCY BETWEEN THE PREDICTED AND THE MEASURED DIFFERENTIAL BORON WORTHS.
(7928 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

+ Plant resumed operation on January 17, 1980.

+LAST IE SITE INSPECTION DATE: December 11-14, 1979

+ INSPECTION REPORT NO: 79-28

SUMMARY

INSPECTION SUMMARY

+ Inspection on October 6 through November 30, (79-24): Review of Operations, Security, Thermal Power Evaluations, Refueling Activities, Procurement, Radiation Protection and Licensee Event Reports (LER's). The inspection involved 134 inspector hours onsite by the NRC Resident Inspector. Of the seven areas inspected, no items of noncompliance were identified.

+ Investigation on November 14-16, (79-25): Six allegations were made that: instructors fired into guards' targets to assist them in obtaining qualifying scores; metal detectors malfunctioned and no alternative procedures were utilized; a waste shipment left the site leaking contamination; personnel who failed the test performed as reactor operators; gaseous radioactive releases were made without NRC being aware; and, a control rod insertion problem may not have been reported to the NRC. The investigation consisted of an examination of records and procedures and interviews with personnel and consisted of 36 investigation hours by two investigators. None of the allegations were substantiated. No items of noncompliance were identified.

+ Inspection on November 19-21, (79-26): Special, announced inspection of radioactive waste systems, including: effluent releases, records and reports of effluents, radioactive waste, response to IE Bulletin 79-19, and followup on previously identified inspection findings. The inspection involved 34 inspector hours onsite by one NRC inspector. There were no items of noncompliance identified in four of the five areas inspected. One item of noncompliance was identified in the area of solid radioactive waste.

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-63/ 03L-0	11/27/79	12/27/79	30 Day LER	While operating in Mode 1, both boric acid transfer pumps were made inoperable for a period of four hours and 40 minutes.
79-64/ 03L-0	12/01/79	12/27/79	30 Day LER	During the investigation of a high steam generator level indication, for channel 1 loop 4, transmitter BLP 141 was found to be 13% out of spec. high.
79-65/ 03T-0	12/24/79	01/16/80	14 Day LER	CAR system seismic design deficiencies.
79-66/ 03L-0	12/18/79	01/16/80	30 Day LER	Glycol containment isolation valve, VCR-10, did not close when its control switch was turned to the closed position.
79-67/ 03L-0	12/21/79	01/16/80	30 Day LER	After the APDMS Fz setpoints were updated it was found that two out of the four thimbles used were not the ones with the lowest uncertainty.

==

+ – Investigation on October 31 through November 1, (79-27): Investigation was conducted into allegations of improper welding tests and record falsification at the licensee's D. C. Cook facility. Investigation examined the licensee's welding certification test procedures and a review of records. Investigation involved 32 hours onsite by one inspector and one investigator. Of the areas investigated, no items of noncompliance were identified.

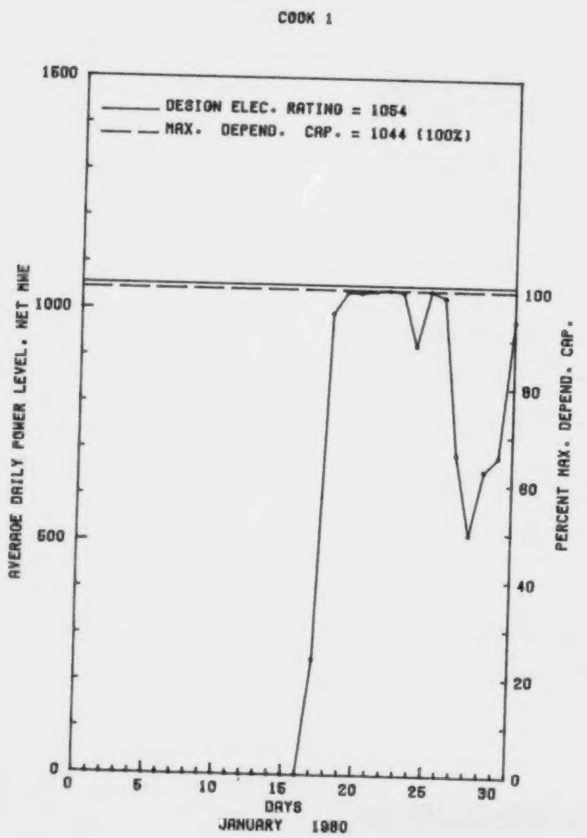
+ = Inspection on December 11-14, (79-28): Routine, announced inspection of shutdown margin determination; isothermal temperature coefficient; power coefficient of reactivity measurement; target axial flux difference; reactivity anomalies. The inspection involved 22 inspector hours onsite by one NRC inspector. Of the five areas inspected, no items of noncompliance or deviations were identified in four areas. One item of noncompliance was identified in one area (failure to update the plant technical data book).

REVIEWED BY OMPA LOEB: T. C. Cintula **DATE:** 3·19·80

REVIEWED BY NRR DOR: DLWigginton **DATE:** 3·21·80

COOK 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-315
2. Utility Contact: W. T. Gillett (616) 465-5901
3. Licensed Thermal Power (MWt): 3250
4. Nameplate Rating (Gross MWe): 1280 x 0.9 = 1152
5. Design Electrical Rating (Net MWe): 1054
6. Maximum Dependable Capacity (Gross MWe): 1080
7. Maximum Dependable Capacity (Net MWe): 1044
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	744.0	44,568.0
12. Hours Reactor Was Critical	346.8	346.8	33,308.5
13. Reactor Reserve Shutdown Hours	.0	.0	463.0
14. Hours Generator On-Line	340.6	340.6	32,423.3
15. Unit Reserve Shutdown Hours	.0	.0	321.0
16. Gross Thermal Energy (MWH)	980,650	980,650	90,046,955
17. Gross Electrical Energy (MWH)	323,510	323,510	29,480,850
18. Net Electrical Energy (MWH)	311,267	311,267	28,308,581
19. Unit Service Factor	45.8	45.8	74.6
20. Unit Availability Factor	45.8	45.8	74.6
21. Unit Capacity Factor (MDC Net)	40.1	40.1	66.5
22. Unit Capacity Factor (DER Net)	39.7	39.7	61.6
23. Unit Forced Outage Rate	54.2	54.2	8.1

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
155	12/24/79	F	396.2	D	4	79-065	SE	SUPORT	OUTAGE CONTINUED FROM PREVIOUS MONTH TO CORRECT DISCREPANCIES IN PIPING SUPPORTS ON SAFETY-RELATE PIPING SYSTEMS AS A RESULT OF IE BULLETIN NO. 79-14. UNIT RETURNED TO SERVICE 01/17/80. TOTAL DURATION OF OUTAGE 570.8 HOURS.
156	01/24/80	F	.0	B	5		ZZ	ZZZZZZ	REACTOR POWER REDUCED TO 60% TO PERMIT REMOVAL OF ONE MAIN FEEDWATER PUMP. ONE FEEDPUMP REMOVED AT A TIME FOR CLEANING OF FEEDPUMP TURBINE CONDENSER TUBE SHEETS.
157	01/27/80	F	7.2	A	3		CH	ZZZZZZ	REACTOR/TURBINE TRIP CAUSED BY STEAM FLOW/FEEDWATER FLOW MISMATCH COINCIDENT WITH LOW LEVEL IN NO. 4 STEAM GENERATOR DUE TO TRIP OF WEST MAIN FEEDPUMP TURBINE. THE FEEDPUMP TURBINE TRIP WAS DUE TO PUMP COUPLING FAILURE.

SUMMARY

THE UNIT REMAINED SHUTDOWN THROUGH 01/16/80 TO CORRECT DESIGN DEFICIENCIES IN THE CONTAINMENT HYDROGEN SKIMMER SYSTEM. LOAD WAS REDUCED ON 01/24/80 FOR REMOVAL OF A FEEDWATER PUMP FOR CLEANING. ON 01/27/80, A FEEDPUMP TURBINE TRIP DUE TO PUMP COUPLING FAILURE CAUSED STEAM FLOW/FEEDWATER FLOW MISMATCH, RESULTING IN A REACTOR/TURBINE TRIP.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

COOK 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Michigan/Berrien 11 miles S of Benton Harbor
2.	Electric Reliability Council	ECAR
3.	Type of Reactor	PWR
4.	Power Level (MWT)	3391
5.	Design Electrical Rating (MWE Net)	1100
6.	Date of Initial Criticality	March 10, 1978
7.	Date of Commercial Operation	July 1, 1978
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10.	Licensee	AEP Corporation Indiana & Michigan Power Co. Indiana & Michigan Electric Co.
11.	Corporate Address	2 Broadway New York, New York 10004
12.	Corporate Contact	John Dolan, VP/Chief Engineer (212) 422 - 4800
13.	Architect/Engineer	American Elec. Power Service Corp.
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Indiana & Michigan Power Co.

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Senior Inspector	D.C. Boyd
19.	IE Resident Inspector	R. Masse (616) 465-6775
20.	Licensing Project Manager	D. Wigginton
21.	Docket Number	50-316
22.	License Number/Date of Issue	DPR-74, 12/23/77
23.	Public Document Room	Maude Preston Palenske Memorial Library 500 Market Street St. Joseph, Michigan 49085

INSPECTION STATUS

ENFORCEMENT STATUS

TECHNICAL SPECIFICATION 6.8.1 REQUIRES IMPLEMENTATION OF PROCEDURES. PMP 6010 RAD.002 STATES IN PART: "EACH 'HIGH RADIATION AREA' DOOR SHALL BE SECURED BY LOCK AND KEY...". CONTRARY TO THE ABOVE, DOOR NO. 370 (TO UNIT 2 CONTAINMENT VESTIBULE AREA) WAS FOUND PROPPED OPEN BY AN ARGON PURGE HOSE BEING USED FOR WELDING IN THE VESTIBULE AREA. THE DOOR WAS UNGUARDED AND UNCONTROLLED.
(7921 INFRACTION)

TECHNICAL SPECIFICATION 6.8.1 REQUIRES IMPLEMENTATION OF PROCEDURES. PMI-2275 STATES IN PART: "THE 'WELDING, BURNING AND GRINDING PERMIT' SHALL BE COMPLETED IN DUPLICATE. AFTER CONCURRENCE BY THE SOE...ONE COPY SHALL BE POSTED IN THE WORK AREA." CONTRARY TO THE ABOVE, THE WELDING JOB REFERENCED IN NO. 1 ABOVE HAD NO EVIDENCE OF AN ACTIVE WELDING PERMIT AT OR NEAR THE WORK AREA.
(7921 INFRACTION)

10 CFR 71.51(A) REQUIRES THAT A LICENSEE "...SHALL ESTABLISH, MAINTAIN AND EXECUTE A QUALITY ASSURANCE PROGRAM SATISFYING EACH OF THE APPLICABLE CRITERIA SPECIFIED IN APPENDIX E, 'QUALITY ASSURANCE CRITERIA FOR SHIPPING PACKAGES FOR RADIOACTIVE MATERIAL'..." IT FURTHER REQUIRES THAT "THE DESCRIPTION OF THE QUALITY ASSURANCE PROGRAM SHALL INCLUDE A DISCUSSION OF WHICH REQUIREMENTS OF APPENDIX E ARE APPLICABLE AND HOW THEY WILL BE SATISFIED." AND THAT "A DESCRIPTION OF THAT PROGRAM SHALL BE FILED, IN ACCORDANCE WITH THIS SECTION, BY JANUARY 1, 1979, WITH THE DIRECTOR, OFFICE OF NUCLEAR MATERIAL SAFETY AND SAFEGUARDS, NUCLEAR REGULATORY COMMISSION, WASHINGTON, D.C. 20555." CONTRARY TO THE ABOVE, NO QUALITY ASSURANCE PROGRAM FOR SHIPPING PACKAGES WAS IN EFFECT AND NO DESCRIPTION OF SUCH A PROGRAM HAD BEEN FILED WITH THE DIRECTOR, NMSS.
(7924 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

+ Plant resumed operation on January 17, 1980.

+LAST IE SITE INSPECTION DATE: October 31 through November 1, 1979

+ INSPECTION REPORT NO: 79-25

SUMMARY

INSPECTION SUMMARY

+ Inspection on October 6 through November 30, (79-21): Review of Operations, Security, Thermal Power Evaluations, Refueling Activities, Procurement, Radiation Protection and Licensee Event Reports (LER's). The inspection involved 134 inspector hours onsite by the NRC Resident Inspector. Of the seven areas inspected, no items of noncompliance were identified in six. In the Operations area, two items of noncompliance were identified as follows: Control of "High Radiation Area" doors and control of ignition sources.

+ Investigation on November 14-16, (79-23): Six allegations were made that: instructors fired into guards' targets to assist them in obtaining qualifying scores; metal detectors malfunctioned and no alternative procedures were utilized; a waste shipment left the site leaking contamination; personnel who failed the test performed as reactor operators; gaseous radioactive releases were made without NRC being aware; and, a control rod insertion problem may not have been reported to the NRC. The investigation consisted of an examination of records and procedures and interviews with personnel and consisted of 36 investigation hours by two investigators. None of the allegations were substantiated. No items of noncompliance were identified.

+ Inspection on November 19-21, (79-24): Special, announced inspection of radioactive waste systems, including: effluent releases, records and reports of effluents, radioactive waste, response to IE Bulletin 79-19, and followup on previously identified inspection findings. The inspection involved 34 inspector hours onsite by one NRC inspector. There were no items of noncompliance identified in four of the five areas inspected. One item of noncompliance was identified in the area of solid radioactive waste.

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-45/ 03L-0	11/13/79	12/12/79	30 Day LER	During refueling the bonnet was removed from a 2 inch hand valve on the blowdown line from No. 2 steam generator while the 1 inch radiographic plug was removed from an elbow in the feedwater line to that steam generator.
79-46/ 03L-0	11/19/79	12/19/79	30 Day LER	With refueling in progress source range neutron flux monitor (N-31) became erratic and was declared inoperable.
79-47/ 03L-0	11/24/79	12/19/79	30 Day LER	During the calibration of grid degraded voltage relays, relay No. 27-1-201-AB dropped out at a value corresponding to 21.9KV.
79-48/ 03L-0	12/01/79	12/27/79	30 Day LER	While in mode 6, with refueling complete, gaseous sample pump tripped.
79-49/ 01T-0	12/18/79	12/31/79	14 Day LER	Significant as-built discrepancy in 1 inch primary coolant piping - IEB 79-14 evaluation.
79-50/ 01T-0	12/17/79	01/11/80	14 Day LER	Items of noncompliance identified during the inspection/evaluation of IEB 79-14.
79-51/ 03L-0	12/17/79	01/16/80	30 Day LER	With reactor in Mode 5 during TS 4.3.2.1.2 containment spray pump discharge valve IMO-220 failed to open.
79-52/ 03L-0	12/18/79	01/16/80	30 Day LER	While in mode 5, source range indication was inoperable for a period of 23 min. this is non-conservative in respect to TS table 3.3-1 Item 6.b.
79-53/ 03L-0	12/26/79	01/17/80	30 Day LER	While performing the type B&C leak test several valves exhibited excessive leakage causing the total as found leak rate to exceed the limit in TS 3.6.1.2.b.
79-54/ 03L-0	12/26/79	01/22/80	30 Day LER	Four relay timers were found to have exceeded the tolerance allowed by TS 4.8.1.1.2.C.2.

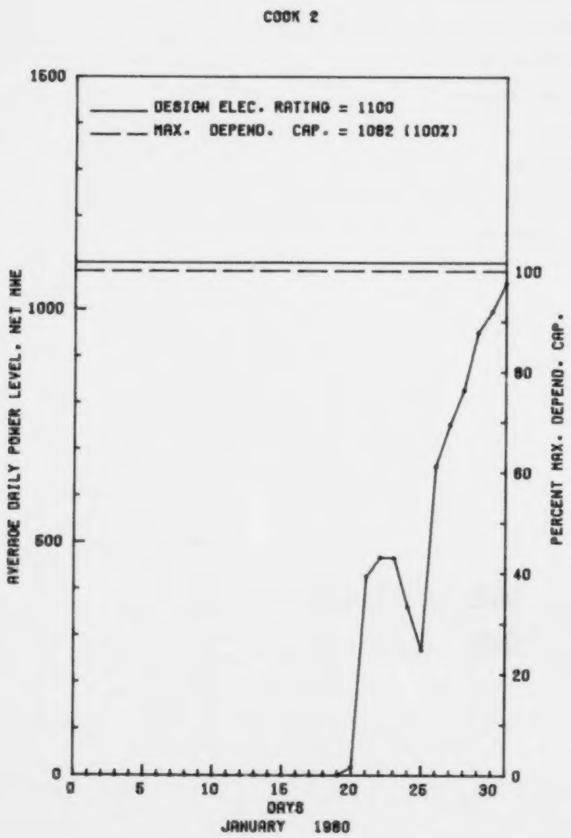
+ Investigation on October 31 through November 1, (79-25): Investigation was conducted into allegations of improper welding tests and record falsification at the licensee's D. C. Cook facility. Investigation examined the licensee's welding certification test procedures and a review of records. Investigation involved 32 hours onsite by one inspector and one investigator. Of the areas investigated, no items of noncompliance were identified.

REVIEWED BY OMPA LOEB: T. C. Cintra **DATE:** 3·19·80

REVIEWED BY NRR DOR: D. L. Wigginton **DATE:** 3·21·80

COOK 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-316
2. Utility Contact: W. T. Gillett (616) 465-5901
3. Licensed Thermal Power (MWt): 3391
4. Nameplate Rating (Gross MWe): 1333 x 0.85 = 1133
5. Design Electrical Rating (Net MWe): 1100
6. Maximum Dependable Capacity (Gross MWe): 1118
7. Maximum Dependable Capacity (Net MWe): 1082
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	744.0	18,254.0
12. Hours Reactor Was Critical	320.8	320.8	11,400.7
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	250.7	250.7	10,752.6
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	584,404	584,404	32,808,784
17. Gross Electrical Energy (MWH)	182,750	182,750	10,337,160
18. Net Electrical Energy (MWH)	173,952	173,952	9,941,364
19. Unit Service Factor	33.7	33.7	67.8
20. Unit Availability Factor	33.7	33.7	67.8
21. Unit Capacity Factor (MDC Net)	21.6	21.6	61.4
22. Unit Capacity Factor (DER Net)	21.3	21.3	60.4
23. Unit Forced Outage Rate	8.9	8.9	13.6

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
64	10/19/79	S	468.7	D	9		ZZ	ZZZZZZ	THE UNIT WAS REMOVED FOR CYCLE 1-II REFUELING OUTAGE ON 10/19/79. THE OUTAGE WAS EXTENDED ON 12/23/79 TO CORRECT DISCREPANCIES FOUND AS A RESULT OF SEISMIC ANALYSIS FOR AS-BUILT SAFETY-RELATED PIPING SYSTEMS PERFORMED IN ACCORDANCE WITH I.E. BULLETIN NO. 79-14. TOTAL DURATION OF OUTAGE 2222.5 HOURS.
65	01/20/80	F	9.6	H	3		ZZ	ZZZZZZ	REACTOR/TURBINE TRIP DUE TO STEAM FLOW/FEEDWATER FLOW MISMATCH COINCIDENT WITH LOW LEVEL IN NO. 2 STEAM GENERATOR. THE TRIP OCCURRED DUE TO DIFFICULTIES WITH STEAM GENERATOR LEVEL CONTROL DURING LOW POWER OPERATION.
66	01/24/80	F	15.0	G	3		ZZ	ZZZZZZ	TURBINE/REACTOR TRIP DUE TO LOW MAIN CONDENSER VACUUM. THE LOW VACUUM CONDITION WAS CAUSED BY OPERATOR VALVING ERROR. UNIT RETURNED TO SERVICE 01/25/80. REACTOR POWER INCREASED TO 90% 01/28/80/
67	01/28/80	F	0	B	5		ZZ	ZZZZZZ	REACTOR POWER REDUCED TO 65% TO REMOVE EAST MAIN FEEDPUMP TURBINE TO CHECK FOR CONDENSER TUBE LEAKS. REACTOR POWER RETURNED TO 90% 01/29/80. REACTOR POWER REACHED 100% FOR THE FIRST TIME ON CYCLE II ON 01/31/80.

SUMMARY

THE UNIT REMAINED SHUTDOWN THROUGH 01/19/80 FOR REFUELING AND TO CORRECT DISCREPANCIES IN SAFETY RELATED PIPING. ON 01/20/80, THE UNIT EXPERIENCED A RECTOR/TURBINE TRIP DUE TO DIFFICULTIES WITH STEAM GENERATOR LEVEL CONTROL DURING LOW POWER OPERATION. A TURBINE/REACTOR TRIP OCCURRED ON 01/24/80 DUE TO LOW MAIN CONDENSER VACUUM CAUSED BY OPERATOR VALVING ERROR. POWER WAS REDUCED ON 01/28/80 TO CHECK THE EAST MAIN FEEDPUMP TURBINE FOR CONDENSER TUBE LEAKS.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

COOPER STATION

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Nebraska/Nemaha 23 miles S of Nebraska City
2.	Electric Reliability Council	MARCA
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2381
5.	Design Electrical Rating (MWE Net)	778
6.	Date of Initial Criticality	February 21, 1974
7.	Date of Commercial Operation	July 1, 1974 (50% rating)
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Missouri River

Utility & Contractor Information

10.	Licensee	Nebraska Public Power District
11.	Corporate Address	P. O. Box 499 Columbus, Nebraska 68601
12.	Corporate Contact	J. M. Pilant, Director Licensing and Quality Assurance
13.	Architect/Engineer	Burns and Roe, Inc.
14.	Nuclear Steam Supply System	GE
15.	Constructor	Burns and Roe, Inc.

Regulatory Information

16.	IE Region responsible	IV
17.	IE Branch Chief	G. L. Madsen
18.	IE Senior Inspector	T. F. Westerman
19.	IE Principal Inspector	G. L. Constable
20.	Licensing Project Manager	V. L. Rooney
21.	Docket Number	50-298
22.	License Number/Date of Issue	DPR-46, 1/18/74
23.	Public Document Room	Auburn Public Library 118 15th Street Auburn, Nebraska 68305

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

FACILITY ITEMS (PLANS AND PROCEDURES):

MANAGERIAL ITEMS:

PLANT STATUS:

UNIT OPERATING IN LOAD FOLLOWING MODE UP TO FULL POWER.
PREPARING FOR REFUELING OUTAGE SCHEDULED TO COMMENCE 3/15/80. +

LAST IE SITE INSPECTION DATE: DECEMBER 3-5, 1979 +

INSPECTION REPORT NO: 50-298/79-17 +

SUMMARY

INSPECTION SUMMARY

+ INSPECTION CONDUCTED DECEMBER 3-5, 1979 (79-17): ROUTINE, UNANNOUNCED INSPECTION OF THE LICENSEE'S EMERGENCY PLANNING PROGRAM INCLUDING COORDINATION WITH OFF-SITE SUPPORT AGENCIES; INSTRUMENTATION AND SUPPLIES; TESTS AND DRILLS; LICENSEE'S INTERNAL AUDIT PROGRAM; EMERGENCY PLAN AND IMPLEMENTING PROCEDURES; EMERGENCY TRAINING; AND OBSERVATION OF AN EMERGENCY DRILL ON DECEMBER 4, 1979. THE INSPECTION INVOLVED 33 ONSITE HOURS BY THREE NRC INSPECTORS. NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

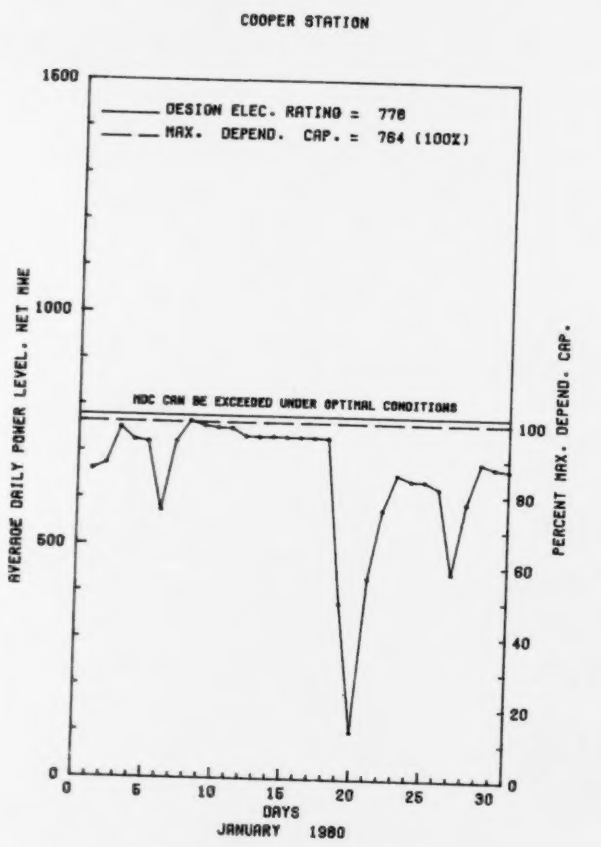
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
79-38 03L-0	11/30/79	12/28/79	SUPPRESSION CHAMBER LEVEL PL-LI-12 FAILURE
79-39 03L-0	11/29/79	12/28/79	SUPPRESSION CHAMBER INBD. VENT ISO VALVE PC-230 MV INOPERABLE FROM CR.
79-40 03L-0	12/3/79	1/2/80	SERVICE WATER BACKWASH LINE LOW WALL THICKNESS
79-41 03L-0	12/6/79	1/8/80	PERSONNEL ERROR - ACCIDENTAL HPCI ISOLATION
79-42 03L-0	12/23/79	1/16/80	RHR PUMP 1D. BKR NOT CHARGED
79-43 03L-0	12/28/79	1/23/80	DEGRADATION OF R.E.C. SYSTEM PIPING
80-01 03L-0	1/2/80	1/11/80	HPCI AUX. OIL PUMP SWITCH FOUND IN PULL-TO-LOCK POSITION
80-02 03L-0	1/3/80	1/21/80	HPCI STOP VALVE STEM BROKEN, VALVE INOP.

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 3-19-80

REVIEWED BY NRR DOR: **DATE:** 3/20/80

COOPER STATION

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-298
2. Utility Contact: P. J. Borer (402) 825-3811
3. Licensed Thermal Power (MWt): 2381
4. Nameplate Rating (Gross MWe): 983 x 0.85 = 836
5. Design Electrical Rating (Net MWe): 778
6. Maximum Dependable Capacity (Gross MWe): 787
7. Maximum Dependable Capacity (Net MWe): 764
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	744.0	48,985.0
12. Hours Reactor Was Critical	729.0	729.0	41,966.3
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	719.7	719.7	41,195.3
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,506,480	1,506,480	77,689,206
17. Gross Electrical Energy (MWH)	500,414	500,414	25,039,262
18. Net Electrical Energy (MWH)	482,289	482,289	24,140,566
19. Unit Service Factor	96.7	96.7	84.1
20. Unit Availability Factor	96.7	96.7	84.1
21. Unit Capacity Factor (MDC Net)	84.8	84.8	64.5
22. Unit Capacity Factor (DER Net)	83.3	83.3	63.3
23. Unit Forced Outage Rate	3.3	3.3	4.6

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUELING AND MAINTENANCE MARCH 15, 1980, 8 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-1	01/06/80	S	.0	H	5				POWER WAS REDUCED TO ALLOW ADJUSTMENT OF THE CONTROL ROD PATTERN.
80-2	01/19/80	F	24.3	A	3				ERRONEOUS MAIN GENERATOR GROUND FAULT INDICATION CAUSED TURBINE TO TRIP AND REACTOR TO SCRAM. PROBLEM CORRECTED AND THE REACTOR AND GENERATOR RETURNED TO NORMAL POWER OPERATION.
80-3	01/27/80	S	.0	H	5				POWER WAS REDUCED TO ALLOW ADJUSTMENT OF THE CONTROL ROD PATTERN.

SUMMARY

OPERATION WAS ROUTINE DURING THE REPORT PERIOD, WITH AN OUTAGE ON 01/19/80 DUE TO AN ERRONEOUS MAIN GENERATOR GROUND FAULT INDICATION CAUSING THE TURBINE TO TRIP AND THE REACTOR TO SCRAM. LOAD REDUCTIONS OCCURRED ON 01/06/80 FOR CONTROL ROD ADJUSTMENT.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

CRYSTAL RIVER 3

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1. Location (state/countv) (nearest population center)	Florida/Citrus 7 miles NW of Crystal River
2. Electric Reliability Council	Southeastern
3. Type of Reactor	PWR
4. Power Level (MWT)	2452
5. Design Electrical Rating (MWE Net)	825
6. Date of Initial Criticality	January 14, 1977
7. Date of Commercial Operation	March 13, 1977
8. Condenser Cooling Method	Once Through
9. Condenser Cooling Water Source	Gulf of Mexico

Utility & Contractor Information

10. Licensee	Florida Power Corporation
11. Corporate Address	3201 34th Street, South St. Petersbury, Florida 33733
12. Corporate Contact	J. Hancock
13. Architect/Engineer	Gilbert Associates, Inc.
14. Nuclear Steam Supply System	B&W
15. Constructor	J. A. Jones Construction Co.

Regulatory Information

16. IE Region responsible	II
17. IE Branch Chief	R. C. Lewis
18. IE Senior Inspector	R. Martin
19. IE Principal Inspector	D. R. Quick
20. Licensing Project Manager	M. B. Fairtile
21. Docket Number	50-302
22. License Number/Date of Issue	DPR-72, 12/3/76
23. Public Document Room	Crystal River Public Library 668 N.W. First Crystal River, Florida 32639

INSPECTION STATUS

ENFORCEMENT STATUS

AS REQUIRED BY TECHNICAL SPECIFICATION 6.8.1.A, WRITTEN PROCEDURES SHALL BE IMPLEMENTED COVERING THE AREA OF EQUIPMENT CONTROL. COMPLIANCE PROCEDURE CP-115 IMPLEMENTS THIS REQUIREMENT AND REQUTRES INDEPENDENT VERIFICATION OF SAFETY RELATED VALVE POSITIONS UPON SYSTEM RESTORATION FOLLOWING MAINTENANCE. CONTRARY TO THE ABOVE EQUIPMENT CLEARANCE NUMBER 10-48 WAS RELEASED ON OCTOBER 22, 1979 WITHOUT HAVING PERFORMED AN INDEPENDENT VERIFICATION OF THE REQUIRED POSITIONS OF ALL VALVES IN THE NUCLEAR SERVICES CLOSED CYCLE COOLING SYSTEM, WITHIN THE BOUNDARIES OF THE CLEARANCE.
(7946 INFRACTION)

AS REQUIRED BY TECHNICAL SPECIFICATION 6.8.1 WRITTEN PROCEDURES SHALL BE ESTABLISHED, IMPLEMENTED AND MAINTAINED. PROCEDURE AI-800 "CONDUCT OF ADMINISTRATIVE SERVICES," SECTION 7.4.5 REQUIRES MEMBERS OF THE EMERGENCY MEDICAL TEAM TO RECEIVE ANNUAL TRAINING. CONTRARY TO THE ABOVE ALL MEMBERS OF EMERGENCY MEDICAL TEAM ARE DEFICIENT IN FIRST-AID TRAINING. ANNUAL TRAINING FOR SHIFT SUPERVISORS NOT RECEIVED DURING YEAR 1979.
(7950 INFRACTION)

AS REQUIRED BY TECHNICAL SPECIFICATION 6.8.1 WRITTEN PROCEDURES SHALL BE ESTABLISHED, IMPLEMENTED AND MAINTAINED. PROCEDURE AI-800 "CONDUCT OF ADMINISTRATIVE SERVICES," SECTION 7.4.5 REQUIRES MEMBERS OF THE EMERGENCY MEDICAL TEAM TO RECEIVE ANNUAL TRAINING. CONTRARY TO THE ABOVE ALL MEMBERS OF EMERGENCY MEDICAL TEAM ARE DEFICIENT IN FIRST-AID TRAINING. ANNUAL TRAINING FOR SHIFT SUPERVISORS NOT RECEIVED DURING YEAR 1979.
(7950 INFRACTION)

BRANCH TECHNICAL POSITION APCSB9.5-1, APPENDIX A, SECTION C.6 REQUIRES IN PART THAT "FIVE EXTINGUISHERS SHOULD BE PROVIDED IN ACCORDANCE WITH GUIDELINES OF NFPA 10 AND 10A. THE FIRE PROTECTION PROGRAM REVIEW, CRYSTAL RIVER 3 DATED JUNE 1977 SECTION 6.E.6 STATES IN PART THAT THE REQUIREMENTS IS MET. FIVE EXTINGUISHERS ARE PROVIDED IN ACCORDANCE WITH NFPA-10. CONTRARY TO THE ABOVE SURVEILLANCE PROCEDURES FOR PORTABLE FIVE EXTINGUISHERS DO NOT REFERENCE NFPA 10 AND DO NOT CONTAIN ALL THE CHECKS AND ACCEPTANCE CRITERIA USED IN NFPA 10.
(7952 DEVIATION)

AS REQUIRED BY TECHNICAL SPECIFICATION 6.12.1.B, AREAS WITH RADIATION LEVELS EXCEEDING 1000 MREM/HR MUST BE PROVIDED WITH DOORS LOCKED TO PREVENT UNAUTHORIZED ENTRY. CONTRARY TO THE ABOVE: (1) ON DECEMBER 17, 1979 THE DOOR TO THE SPENT RESIN TRANSFER PUMP AREA WAS NOT CLOSED. THIS PROVIDED ACCESS TO THE SPENT RESIN CATCH TANK THRU A CRAWL WAY. RADIATION LEVELS IN THE CRAWL WAY WERE MEASURED AT 2000 MREM/HR; AND (2) ON DECEMBER 17, 1979 DOORS CONTROLLING ACCESS TO THE EVAPORATOR CONDENSATE DEMINERALIZER AREA AND THE MAKE UP PREFILTER AREA WERE INADEQUATE TO PREVENT UNAUTHORIZED ENTRY. THE LOCK MECHANISMS COULD BE EASILY DEFEATED TO GAIN ACCESS TO THESE AREAS. SURVEYS CONDUCTED ON DECEMBER 19 INDICATED GENERAL AREA RADIATION LEVELS OF 1500 MREM/HR (MAKE UP PREFILTER AREA) AND 2500 MREM/HR (DEMINERALIZER AREA).
(7954 INFRACTION)

AS REQUIRED BY 10 CFR 20.203(E), AREAS WHERE RADIOACTIVE MATERIALS IN EXCESS OF 10 TIMES 10 CFR 20 APPENDIX C QUANTITIES ARE STORED MUST BE POSTED WITH A SIGN OR SIGNS BEARING THE RADIATION CAUTION SYMBOL AND THE WORDS: CAUTION, RADIOACTIVE MATERIAL(S). ALSO, AS REQUIRED BY 10 CFR 20.203(E), CONTAINERS CONTAINING RADIOACTIVE MATERIALS IN EXCESS OF 10 CFR 20 APPENDIX C QUANTITIES MUST HAVE A LABEL IDENTIFYING THE RADIOACTIVE CONTENTS. THIS LABEL MUST BEAR THE RADIATION CAUTION SYMBOL, THE WORDS: "DANGER, RADIOACTIVE MATERIAL," OR "CAUTION, RADIOACTIVE MATERIAL" AND MUST PROVIDE SUFFICIENT INFORMATION TO PERMIT INDIVIDUALS HANDLING OR USING THE CONTAINERS, OR WORKING IN THE VICINITY THEREOF, TO TAKE PRECAUTIONS TO AVOID OR MINIMIZE EXPOSURES. CONTRARY TO THE ABOVE, ON DECEMBER 17, 1979, APPROXIMATELY 50 BAGS OF CONTAMINATED WASTE AWAITING COMPACTION AND SHIPMENT AS RADIOACTIVE WASTE WERE IN A HOLDING AREA. THESE BAGS HAD NO TAG AFFIXED TO INDICATE CONTAMINATION OR RADIATION LEVELS. QUANTITIES OF RADIOACTIVE MATERIAL IN EXCESS OF APPENDIX C QUANTITIES WERE PRESENT.
(7954 DEFICIENCY)

AS REQUIRED BY TECHNICAL SPECIFICATION 6.8, PROCEDURES ADDRESSING AREAS OUTLINED IN REGULATORY GUIDE 1.33 (1972), QUALITY ASSURANCE PROGRAM REQUIREMENTS, APPENDIX A, MUST BE FOLLOWED. SECTION I.5, APPENDIX A REQUIRES PROCEDURES FOR CONTROL OF MODIFICATIONS. FACILITY COMPLIANCE PROCEDURE CP-114, "PROCEDURE FOR PREPARATION AND CONTROL OF PERMANENT MODIFICATIONS, TEMPORARY MODIFICATIONS, DEVIATIONS, AND MAR FUNCTIONAL TEST PROCEDURES" IMPLEMENTS THIS REQUIREMENT. CP-114, SECTION 1.1 STATES, "ANY CHANGE FROM ORIGINAL DESIGN REQUIREMENTS IS A MODIFICATION AND MUST BE CONTROLLED AND RECEIVE DESIGN REVIEW EQUIVALENT TO THAT IMPOSED ON THE ORIGINAL DESIGN." CONTRARY TO THE ABOVE, ON DECEMBER 18, 1979, SIX LEAD BLANKETS, (ABOUT 30 POUNDS EACH) WERE FOUND ON MAKE UP SYSTEM PIPING NEAR VALVE MOV-305 WITH NO ANALYSIS OR CONTROL OVER THE AMOUNT OF LEAD INSTALLED AND ITS EFFECT ON THE SEISMIC QUALIFICATION OF THE SYSTEM. THIS PIPING IS A SEISMIC 1 SAFETY SYSTEM.
(7954 INFRACTION)

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-107/ 03L-0	11/20/79	12/18/79	30 DAY LER	CHEMICAL ADDITION CONTAINMENT ISOLATION VALVE CAV-2 INOPERABLE.
79-108/ 03L-0	12/01/79	12/21/79	30 DAY LER	EMERGENCY DIESEL STANDBY COOLING PUMP DJP-4 INOPERABLE.

INSPECTION STATUS

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

PLANT STATUS:

NORMAL OPERATION AT 100% POWER - NO APPARENT MAJOR PROBLEMS.

* LAST IE SITE INSPECTION DATE: FEBRUARY 4-8, 1980

* INSPECTION REPORT NO: 50-302/80-05
SUMMARY

INSPECTION SUMMARY

* INSPECTION OCTOBER 29 - NOVEMBER 2 (79-46): THIS ROUTINE, UNANNOUNCED
INSPECTION INVOLVED 32 INSPECTOR-HOURS ONSITE IN THE AREAS OF REVIEW
OF OPERATIONS, REVIEW OF IE BULLETINS AND CIRCULARS, FOLLOWUP ON PRE-
VIOUSLY IDENTIFIED ITEMS, AND A PLANT TOUR. OF THE FOUR AREAS INSPECTED,
NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN THREE AREAS;
ONE ITEM OF NONCOMPLIANCE WAS FOUND IN ONE AREA (INFRACTION - FAILURE
TO PERFORM INDEPENDENT VERIFICATION OF SAFETY-RELATED VALVE POSITIONS
FOLLOWING MAINTENANCE).

INSPECTION DECEMBER 3-7 (79-50): THIS ROUTINE, ANNOUNCED INSPECTION
INVOLVED 94 INSPECTOR-HOURS ON SITE AND OFFSITE IN THE AREAS OF
COORDINATION WITH OFFSITE SUPPORT AGENCIES; EMERGENCY FACILITIES,
EQUIPMENT AND PROCEDURES; MEANS FOR DETERMINING A RELEASE; EMERGENCY
TRAINING; FIRE BRIGADE ORGANIZATION AND TRAINING; EMERGENCY ORGANIZA-
TION; IE BULLETIN 79-18; EMERGENCY DRILLS; LICENSEE QA AUDITS AND
FOLLOWUP ON PREVIOUS INSPECTION FINDINGS. OF THE TEN AREAS INSPECTED,
NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN NINE AREAS;
ONE APPARENT ITEM OF NONCOMPLIANCE WAS FOUND IN ONE AREA (FAILURE TO
MAINTAIN TRAINING PROGRAM AS REQUIRED BY ADMINISTRATIVE PROCEDURE A.I.
800).

INSPECTION NOVEMBER 26-29 (79-52): THIS ROUTINE, UNANNOUNCED INSPECTION
INVOLVED 28 INSPECTOR-HOURS ON SITE IN THE AREAS OF PORTABLE FIRE EXTIN-
GUISHER SURVEILLANCE AND COMPLIANCE WITH LIMITING CONDITIONS FOR OPERATION.
OF THE TWO AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIA-
TIONS WERE IDENTIFIED IN ONE AREA; ONE APPARENT DEVIATION WAS FOUND IN
ONE AREA (DEVIATION - PORTABLE FIRE EXTINGUISHER SURVEILLANCE PROCEDURES
DO NOT FOLLOW NFPA-10).

INSPECTION DECEMBER 3-7 (79-53): THIS ROUTINE, ANNOUNCED INSPECTION
INVOLVED 31 INSPECTOR-HOURS ON SITE IN THE AREAS OF OPERATIONS REVIEW
AND OPEN ITEMS. OF THE TWO AREAS INSPECTED, NO APPARENT ITEMS OF
NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

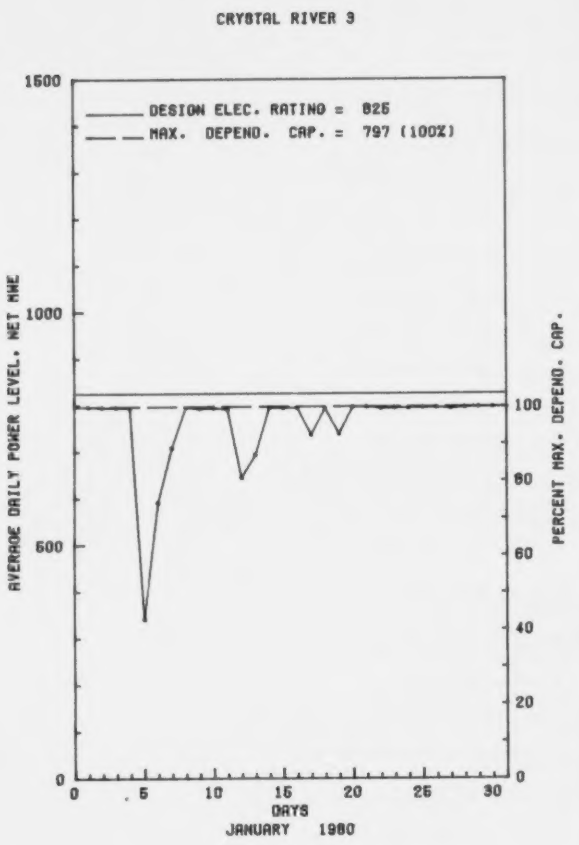
INSPECTION DECEMBER 17-19 (79-54): THIS ROUTINE, UNANNOUNCED INSPECTION
INVOLVED 50 INSPECTOR-HOURS ON SITE IN THE AREAS OF RADIOACTIVE MATERIAL
TRANSPORTATION, HIGH RADIATION AREA CONTROL, AND CONTAMINATED MATERIAL
CONTROLS AND FACILITY TOURS. OF THE FOUR AREAS INSPECTED, NO APPARENT
ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN ONE AREA; THREE
APPARENT ITEMS OF NONCOMPLIANCE WERE FOUND IN THREE AREAS (INFRACTION -
FAILURE TO LOCK OR ADEQUATELY LOCK HIGH RADIATION AREAS; INFRACTION -
FAILURE TO REVIEW SEISMIC EFFECTS OF LEAD SHIELDING OF MAKE UP SYSTEM
PIPING; DEFICIENCY - POSTING AND LABELING OF CONTAMINATED MATERIAL).

REVIEWED BY OMPA LOEB: **DATE:** 3-19-80

REVIEWED BY NRR DOR: Morton B. Fairtile **DATE:** 3/20/80

CRYSTAL RIVER 3

AVERAGE POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-302
2. Utility Contact: W. A. Stephenson (904) 795-6486
3. Licensed Thermal Power (MWt): 2452
4. Nameplate Rating (Gross MWe): 989 × 0.9 = 890
5. Design Electrical Rating (Net MWe): 825
6. Maximum Dependable Capacity (Gross MWe): 835
7. Maximum Dependable Capacity (Net MWe): 797
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	744.0	25,320.0
12. Hours Reactor Was Critical	733.0	733.0	15,883.2
13. Reactor Reserve Shutdown Hours	.0	.0	1,116.9
14. Hours Generator On-Line	729.4	729.4	15,429.7
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,740,359	1,740,359	34,057,270
17. Gross Electrical Energy (MWH)	599,938	599,938	11,571,706
18. Net Electrical Energy (MWH)	571,442	571,442	10,963,082
19. Unit Service Factor	98.0	98.0	60.9
20. Unit Availability Factor	98.0	98.0	60.9
21. Unit Capacity Factor (MDC Net)	96.4	96.4	54.3
22. Unit Capacity Factor (DER Net)	93.1	93.1	52.5
23. Unit Forced Outage Rate	.0	.0	34.4

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
PORV & REFUELING - 3/29/80 - 49 DAYS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-1	01/05/80	S	14.6	B	1		CB	PUMPXX	SHUTDOWN TO RESTORE RCP OIL LEVEL. NO LEAKS OR PROBLEMS WERE DISCOVERED.
80-2	01/07/80	F	4.0	B	5		CH	PUMPXX	REDUCED POWER TO 46% FP TO REPAIR A FWP OIL LEAK.
80-3	01/11/80	S	38.0	B	5		HC	HTEXCH	REDUCED POWER TO 80% FOR CONDENSER WATERBOX CLEANING.
80-4	01/17/80	F	4.0	B	5		CC	VALVEX	REDUCED POWER TO 65% FOR REPAIR OF A THROTTLE VALVE.
80-5	01/19/80	F	11.3	A	5		HH	INSTRU	REDUCED POWER TO 95% DUE TO LEVEL PROBLEMS WITH FWHE 5A. THE LEVEL PROBLEMS WERE CAUSED BY A MISCALIBRATED TRANSMITTER. THE LINE WAS BLOWN DOWN AND THE TRANSMITTER RECALIBRATED.

SUMMARY

OPERATION WAS AT NEAR FULL POWER DURING THE REPORT PERIOD EXCEPT FOR AN OUTAGE ON 01/05/80 TO RESTORE REACTOR COOLANT PUMP OIL LEVEL. BRIEF REDUCTIONS WERE NECESSARY TO CLEAN CONDENSER WATERBOXES, REPAIR A THROTTLE VALVE AND A FEEDWATER PUMP OIL LEAK, AND INVESTIGATE PROBLEMS WITH THE 5A FEEDWATER HEATER.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

DAVIS-BESSE 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1. Location (state/county) (nearest population center): Ohio/Ottawa, 21 miles E of Toledo
2. Electric Reliability Council: ECAR
3. Type of Reactor: PWR
4. Power Level (MWT): 2772
5. Design Electrical Rating (MWE Net): 906
6. Date of Initial Criticality: August 12, 1977
7. Date of Commercial Operation: 11/21/77 (25%); 12/19/77 (40%) 01/23/78 (75%); 07/31/78 (100%)
8. Condenser Cooling Method: Cooling Tower
9. Condenser Cooling Water Source: Lake Erie

Utility & Contractor Information

10. Licensee: Toledo Edison Co.
11. Corporate Address: Edison Plaza, 300 Madison Avenue, Toledo, Ohio 43652
12. Corporate Contact: R. P. Crouse, V. P., Nuclear (419)259-5221
13. Architect/Engineer: Bechtel
14. Nuclear Steam Supply System: B&W
15. Constructor: Bechtel

Regulatory Information

16. IE Region responsible: III
17. IE Branch Chief: R. Heishman
18. IE Senior Inspector: R. F, Warnick
19. IE Principal Inspector: L. Reyes
20. Licensing Project Manager: D. Garner
21. Docket Number: 50-346
22. License Number/Date of Issue: NPF-3, 4/22/77
23. Public Document Room: Ida Rupp Public Library, 310 Madison Street, Port Clinton, OH 43452

INSPECTION STATUS

ENFORCEMENT STATUS

TECHNICAL SPECIFICATION 3.4.1.A, REQUIRES THAT IN MODES 1 AND 2, WITH ONE REACTOR COOLANT PUMP NOT IN OPERATION, STARTUP AND POWER OPERATION MAY BE INITIATED AND MAY PROCEED PROVIDED THERMAL POWER IS RESTRICTED TO LESS THAN 78.3% OF RATED THERMAL POWER AND WITHIN 4 HOURS THE SETPOINTS FOR THE FOLLOWING TRIPS HAVE BEEN REDUCED TO THE VALUES SPECIFIED IN SPECIFICATION 2.2.1 FOR OPERATION WITH THREE REACTOR COOLANT PUMPS OPERATING: (1) HIGH FLUX AND (2) FLUX-FLUX-FLOW. THE SURVEILLANCE PORTION OF THAT SPECIFICATION REQUIRES THAT THE REACTOR PROTECTIVE INSTRUMENTATION CHANNELS SPECIFIED BE VERIFIED TO HAVE HAD THEIR TRIP SETPOINTS CHANGED TO THE VALUES SPECIFIED IN SPECIFICATION 2.2.1 FOR THE APPLICABLE NUMBER OF REACTOR COOLANT PUMPS OPERATING...: (A) WITHIN 4 HOURS AFTER SWITCHING TO A DIFFERENT PUMP COMBINATION IF THE SWITCH IS MADE WHILE OPERATING. CONTRARY TO THE ABOVE ON NOVEMBER 26, 1979, WHILE IN MODE 1, REACTOR COOLANT PUMP 1-2 WAS TRIPPED AND THE HIGH FLUX TRIP SETPOINT WAS NOT CHANGED TO THE VALUE SPECIFIED IN TABLE 2.2.1 WITHIN 4 HOURS AFTER THE CHANGE TO 3 RCP OPERATION. THE FLUX-FLUX-FLOW TRIP SETPOINT WAS NOT VERIFIED TO HAVE ITS SETPOINT CHANGED TO THE VALUE SPECIFIED IN TS 2.2.1 WITHIN 4 HOURS AFTER THE CHANGE TO 3 RCP OPERATION.
(7929 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

- Routine operation since return to operation on January 7, 1980 from the December outage to repair and replace hot leg RTD's.

+LAST IE SITE INSPECTION DATE: October 15-20; November 1-2, 15-16 and 28, 1979

- INSPECTION REPORT NO: 79-32

SUMMARY

INSPECTION SUMMARY

- Inspection on November 1-2, 5-10, 12-16, 19-21 and 26-30, (79-29): A routine, unannounced inspection of followup of previous inspection findings, plant operations, physical protection (security organization, physical barriers, access control, communications), followup of licensee event reports, followup of IE Bulletins and NRR generic letter on Containment Purging. The inspection involved 140 inspector hours onsite by the Resident Inspector. Of the nine areas inspected, no items of noncompliance or deviations were found in eight areas; one item of apparent noncompliance was identified in the other area (failure to verify change in High Flux and Flux-Flux-Flow trip setpoints).

- Inspection on October 15-20; November 1-2, 15-16 and 28, (79-32): Special, unannounced inspection associated with the reactor trip and subsequent loss of offsite power that occurred October 15, 1979; followup on corrective action and the tripping of a reactor coolant pump on October 25, 1979. The inspection involved 153 inspector hours onsite by four NRC inspectors. An Immediate Action Letter was issued October 17, 1979 confirming corrective action on five equipment problems prior to the unit startup. No items of noncompliance were identified.

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 3-19-80

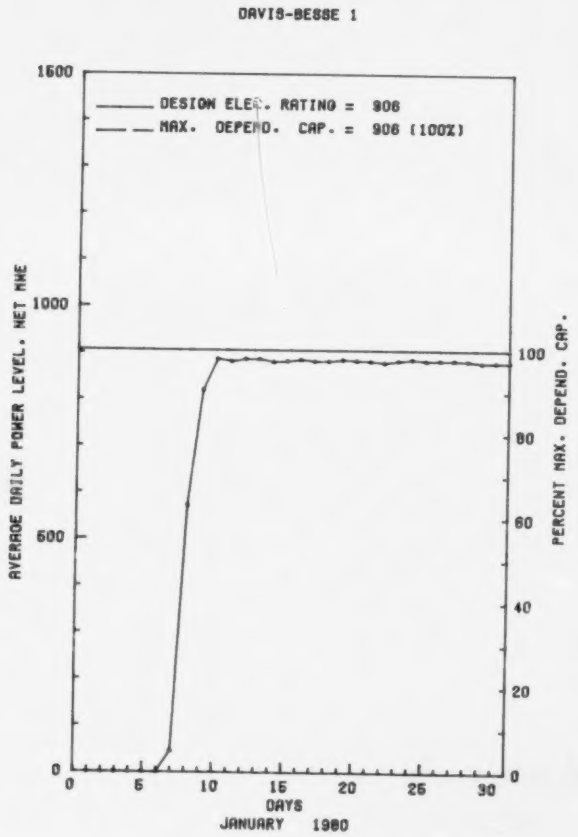
REVIEWED BY NRR DOR: **DATE:** 3/21/80

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-112/ 03L-0	11/19/79	12/18/79	30 Day LER	Valve MS106 failed to open during surveillance testing.
79-113/ 03L-0	11/21/79	12/18/79	30 Day LER	Station personnel failed to perform hourly surveillance requirements of TS when incore instrumentation was inoperable.
79-114/ 03L-0	11/22/79	12/20/79	30 Day LER	RCS indicated flow was below the requirement of TS.
79-115/ 03L-0	11/22/79	12/20/79	30 Day LER	Remote contact data typer became inoperable removing control room indication of fire alarms.
79-116/ 03L-0	11/23/79	12/20/79	30 Day LER	Absolute position indication (API) declared inoperable due to fluctuating signals.
79-117/ 03L-0	11/26/79	12/21/79	30 Day LER	Snubber on decay heat string found upside down rendering snubber inoperable.
79-118/ 03L-0	11/26/79	12/21/79	30 Day LER	A low oil level alarm for RCP was received.
79-119/ 03L-0	12/03/79	12/21/79	30 Day LER	Post-accident radiation monitors could have had the high setpoint for release set higher than allowed by TS.
79-120/ 03L-0	11/29/79	12/26/79	30 Day LER	Numerous asymmetric alarms & faults from control rod group were received.
79-121/ 03L-0	11/30/79	12/28/79	30 Day LER	Group rod indicated about 8% lower than the group average.
79-122/ 03L-0	12/01/79	12/28/79	30 Day LER	Reactor Protection System (RPS) channel indication was failed low.
79-123/ 03L-0	12/07/79	01/05/80	30 Day LER	Service water pump strainer motor burned.
79-124/ 03L-0	12/08/79	01/05/80	30 Day LER	Component cooling water pump would not start from the control room.
79-125/ 03L-0	12/08/79	01/05/80	30 Day LER	Decay heat cooler CCW outlet valve would not stroke from control room switch.
79-126/ 03L-0	12/09/79	01/05/80	30 Day LER	DG experienced load swings as the load increased and decreased.
79-127/ 03L-0	12/10/79	01/05/80	30 Day LER	IBM system 7 contact data logger failed.
79-128/ 03L-0	12/10/79	01/05/80	30 Day LER	Emergency ventilation system charcoal filter bank failed to meet efficiency limit as required by TS.
79-129/ 03L-0	12/11/79	01/08/80	30 Day LER	Control room emergency ventilation system charcoal filter bank failed to meet required efficiency by TS.
79-130/ 03L-0	12/27/79	01/08/80	30 Day LER	Insufficient clearance between RCS hot leg and shims of pipe whip restraints.
79-131/ 03L-0	12/12/79	01/09/80	30 Day LER	Steam and feedwater rupture control system failed.
79-132/ 01T-0	12/31/79	01/11/80	14 Day LER	Procedures governing nuclear instrumentation found to be inconsistent.
79-133/ 03L-0	12/19/79	01/15/80	30 Day LER	Seismic support on high pressure injection line exceeded slenderness design per IEB 79-02.
79-134/ 03L-0	12/29/79	01/24/80	30 Day LER	IBM system 7 contact data logger failed.
80-01/ 01T-0	01/03/80	01/15/80	14 Day LER	Safety features actuation system tripped.

DAVIS-BESSE 1

AVERAGE POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-346
2. Utility Contact: E. Caba (419) 259-5000 x236
3. Licensed Thermal Power (MWt): 2772
4. Nameplate Rating (Gross MWe): 1069 x 0.9 = 962
5. Design Electrical Rating (Net MWe): 906
6. Maximum Dependable Capacity (Gross MWe): 906
7. Maximum Dependable Capacity (Net MWe): 906
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	744.0	21,269.0
12. Hours Reactor Was Critical	624.1	624.1	11,588.3
13. Reactor Reserve Shutdown Hours	0	0	2,875.8
14. Hours Generator On-Line	593.1	593.1	10,467.8
15. Unit Reserve Shutdown Hours	0	0	1,728.2
16. Gross Thermal Energy (MWH)	1,570,427	1,570,427	21,769,934
17. Gross Electrical Energy (MWH)	527,049	527,049	7,250,560
18. Net Electrical Energy (MWH)	496,637	496,637	6,667,215
19. Unit Service Factor	79.7	79.7	50.6
20. Unit Availability Factor	79.7	79.7	59.6
21. Unit Capacity Factor (MDC Net)	73.7	73.7	37.6
22. Unit Capacity Factor (DER Net)	73.7	73.7	37.6
23. Unit Forced Outage Rate	20.3	20.3	27.2

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUELING OUTAGE, MARCH, 1980 - 12 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
21	11/30/79	F	150.9	B	4				STARTUP IN PROGRESS AFTER OUTAGE WAS EXTENDED TO REPAIR RTD RC3A3.

SUMMARY

THE UNIT RETURNED TO POWER ON 01/07/80 FROM THE 11/79 OUTAGE TO INVESTIGATE THE MOTOR BEARINGS OIL LEVEL ALARM ON REACTOR COOLANT PUMP 1-2, AND TO FIX CONTROL ROD ABSOLUTE POSITION INDICATORS. OPERATION WAS AT NEAR FULL POWER THE REMAINDER OF THE REPORT PERIOD.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

DRESDEN 1

FACILITY DATA

Facility Description

1. Location (state/county) (nearest population center)	Illinois/Grundy 9 miles E of Morris
2. Electric Reliability Council	Main
3. Type of Reactor	BWR
4. Power Level (MWT)	700
5. Design Electrical Rating (MWE Net)	200
6. Date of Initial Criticality	October 15, 1959
7. Date of Commercial Operation	July 4, 1960
8. Condenser Cooling Method	Once Through
9. Condenser Cooling Water Source	Kankakee River

Utility & Contractor Information

10. Licensee	Commonwealth Edison Co.
11. Corporate Address	P. O. Box 767 Chicago, Illinois 60690
12. Corporate Contact	C. Reed
13. Architect/Engineer	Bechtel
14. Nuclear Steam Supply System	GE
15. Constructor	Bechtel

Regulatory Information

16. IE Region responsible	III
17. IE Branch Chief	R. Heishman
18. IE Senior Inspector	R. Spessard
19. IE Resident Inspector	J. L. Barker (815)-942-9267
20. Licensing Project Manager	S. J. Nowicki
21. Docket Number	50-010
22. License Number/Date of Issue	DPR-2 issued 9/28/59
23. Public Document Room	Morris Public Library 604 Liberty Street Morris, Illinois 60451

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

MANAGERIAL ITEMS

* A management meeting was held on December 4, 1979, with corporate management personnel in the Commonwealth Edison Offices in Chicago, Illinois to discuss recent personnel errors experienced at the Dresden Station. (79-24)

PLANT STATUS

Unit 1 shutdown on October 31, 1978 for an extended outage to upgrade ECCS, chemical cleaning and refueling. Outage expected to last a minimum of 18 months.

*LAST IE SITE INSPECTION DATE: November 5 through December 4, 1979

* INSPECTION REPORT NO: 79-24

SUMMARY

INSPECTION SUMMARY

* Inspection on November 5 through December 4, (79-24): Routine, unannounced inspection of plant operations; physical protection - security organization, physical barriers, access control (identification, authorization, badging, search, and escorting), and communications; and, corporate management meeting. The inspection involved 53 inspector hours onsite by one NRC inspector and three manhours at the corporate office during a management meeting by three NRC personnel. No items of noncompliance were identified.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
NONE				

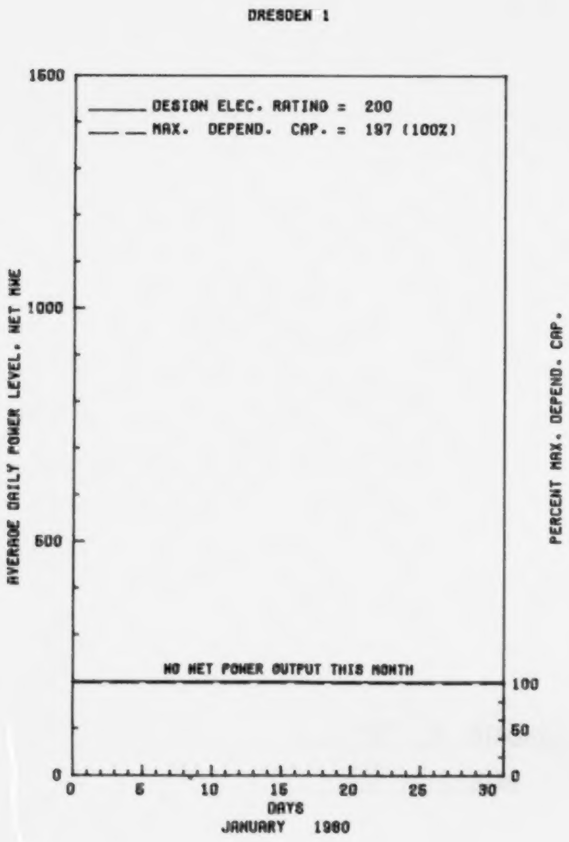
REVIEWED BY OMPA LOEB: B Muranaka DATE: 3/19/80

REVIEWED BY NRR DOR: S. J. Nowicki DATE: 3-20-80

GPO 871-667

DRESDEN 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-010
2. Utility Contact: S. A. Cieszkiewicz (815) 942-2920
3. Licensed Thermal Power (MWt): 700
4. Nameplate Rating (Gross MWe): 245.5 x 0.85 = 209
5. Design Electrical Rating (Net MWe): 200
6. Maximum Dependable Capacity (Gross MWe): 205
7. Maximum Dependable Capacity (Net MWe): 197
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	744.0	173,544.0
12. Hours Reactor Was Critical	.0	.0	113,153.9
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	.0	.0	111,196.2
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	0	56,014,747
17. Gross Electrical Energy (MWH)	0	0	16,799,254
18. Net Electrical Energy (MWH)	-875	-875	15,808,952
19. Unit Service Factor	.0	.0	64.1
20. Unit Availability Factor	.0	.0	64.1
21. Unit Capacity Factor (MDC Net)	.0	.0	46.2
22. Unit Capacity Factor (DER Net)	.0	.0	45.5
23. Unit Forced Outage Rate	.0	.0	11.2

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: INDEFINITE

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
8	10/31/78	S	744.0	C	4				REFUEL AND CHEMICAL CLEANING OUTAGE.

SUMMARY

THE UNIT WAS SHUTDOWN THROUGHOUT THE REPORT PERIOD FOR REFUELING AND CHEMICAL CLEANING.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

DRESDEN 2

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Illinois/Grundy 9 miles E of Morris
2.	Electric Reliability Council	Main
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2527
5.	Design Electrical Rating (MWE Net)	794
6.	Date of Initial Criticality	January 7, 1970
7.	Date of Commercial Operation	JULY, 1970
8.	Condenser Cooling Method	Cooling Lake
9.	Condenser Cooling Water Source	Kankakee and Des Plaines Rivers

Utility & Contractor Information

10.	Licensee	Commonwealth Edison Co.
11.	Corporate Address	P. O. Box 767 Chicago, Illinois 60690
12.	Corporate Contact	C. Reed
13.	Architect/Engineer	Sargent & Lundy
14.	Nuclear Steam Supply System	GE
15.	Constructor	United Engineers & Constructors, Inc.

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Senior Inspector	R. Spessard
19.	IE Resident Inspector	J. L. Barker (815)-942-9267
20.	Licensing Project Manager	S. J. Nowicki
21.	Docket Number	50-237
22.	License Number/Date of Issue	DPR-19 issued 12/22/69
23.	Public Document Room	Morris Public Library 604 Liberty Street Morris, Illinois 60451

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

MANAGERIAL ITEMS

- A management meeting was held on December 4, 1979, with corporate management personnel in the Commonwealth Edison Offices in Chicago, Illinois to discuss recent personnel errors experienced at the Dresden Station. (79-28)

PLANT STATUS

Unit operating routinely.

+LAST IE SITE INSPECTION DATE: November 5 through December 4, 1979

+ INSPECTION REPORT NO: 79-28

SUMMARY

INSPECTION SUMMARY

- Inspection on September 26-27; October 3-4, 22-23 and November 7-9, (79-22): Follow up to IE Bulletin No. 79-02, pipe support base plate designs using concrete expansion anchor bolts; review of test program and procedures; observation of testing anchor bolts; and review of quality records. The inspection involved 68 inspector hours onsite by one NRC inspector. No items of noncompliance were identified in the areas inspected.
- Inspection on November 5 through December 4, (79-28): Routine, unannounced inspection of plant operations; physical protection - security organization, physical barriers, access control (identification, authorization, badging, search, and escorting), and communications; and, corporate management meeting. The inspection involved 53 inspector hours onsite by one NRC inspector and three manhours at the corporate office during a management meeting by three NRC personnel. No items of noncompliance were identified.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-64/ 03L-0	11/24/79	12/24/79	30 Day LER	Torus to drywell vacuum breaker 2-1601-32F.
79-65/ 03L-0	11/30/79	12/30/79	30 Day LER	"B" SBGT system control valve failed to open.
79-66/ 03L-0	12/13/79	01/11/80	30 Day LER	Greater than 0.7 curies in "A" waste sample tank.
79-67/ 03L-0	12/18/79	01/17/80	30 Day LER	2/3 diesel generator declared inoperable.
79-68/ 03L-0	12/29/79	01/23/80	30 Day LER	Integrity of fire stop in vicinity of Bus 28-2 inadequate.

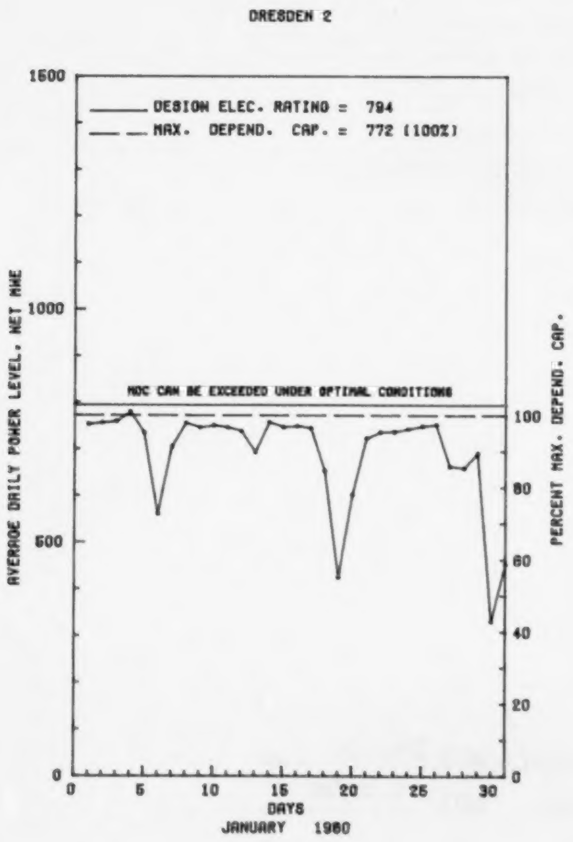
==

REVIEWED BY OMPA LOEB: BMmanela **DATE:** 3/19/80

REVIEWED BY NRR DOR: S. J. Nowicki **DATE:** 3-20-80

DRESDEN 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-237
2. Utility Contact: S. A. Cieszkiewicz (815) 942-2920 x263
3. Licensed Thermal Power (MWt): 2527
4. Nameplate Rating (Gross MWe): 920 x 0.9 = 828
5. Design Electrical Rating (Net MWe): 794
6. Maximum Dependable Capacity (Gross MWe): 812
7. Maximum Dependable Capacity (Net MWe): 772
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	744.0	85,200.0
12. Hours Reactor Was Critical	734.7	734.7	65,471.4
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	729.7	729.7	61,992.4
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,669,987	1,669,987	123,311,819
17. Gross Electrical Energy (MWH)	537,321	537,321	39,543,962
18. Net Electrical Energy (MWH)	511,320	511,320	37,399,432
19. Unit Service Factor	98.1	98.1	72.8
20. Unit Availability Factor	98.1	98.1	72.8
21. Unit Capacity Factor (MDC Net)	89.0	89.0	56.9
22. Unit Capacity Factor (DER Net)	86.6	86.6	55.3
23. Unit Forced Outage Rate	1.9	1.9	13.4

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
1	01/30/80	F	14.3	G	1				THE UNIT WAS INADVERTENTLY MANUALLY SCRAMMED UPON RECEIVING A HALF-SCRAM SIGNAL ON REACTOR PROTECTION SYSTEM CHANNEL B.

SUMMARY

OPERATION WAS ROUTINE DURING THE REPORT PERIOD, WITH A MANUAL SCRAM ON 01/30/80 AFTER RECEIVING A HALF-SCRAM SIGNAL ON THE REACTOR PROTECTION SYSTEM CHANNEL B.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

DRESDEN 3

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Illinois/Grundy 9 miles E of Morris
2.	Electric Reliability Council	Main
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2527
5.	Design Electrical Rating (MWE Net)	794
6.	Date of Initial Criticality	January 31, 1971
7.	Date of Commercial Operation	November 16, 1971
8.	Condenser Cooling Method	Cooling Lake
9.	Condenser Cooling Water Source	Kankakee and Des Plaines Rivers

Utility & Contractor Information

10.	Licensee	Commonwealth Edison Co.
11.	Corporate Address	P. O. Box 767 Chicago, Illinois 60690
12.	Corporate Contact	D. L. Peoples
13.	Architect/Engineer	Sargent & Lundy
14.	Nuclear Steam Supply System	GE
15.	Constructor	United Engineers & Constructors, Inc.

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Senior Inspector	R. Spessard
19.	IE Resident Inspector	J. L. Barker 815-942-9267
20.	Licensing Project Manager	R. Bevan
21.	Docket Number	50-249
22.	License Number/Date of Issue	DPR-25 issued 1/12/71
23.	Public Document Room	Morris Public Library 604 Liberty Street Morris, Illinois 60451

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

MANAGERIAL ITEMS

+ A management meeting was held on December 4, 1979, with corporate management personnel in the Commonwealth Edison Offices in Chicago, Illinois to discuss recent personnel errors experienced at the Dresden Station. (79-26)

PLANT STATUS

The unit is operating at 70% power (Administrative Restriction Imposed by licensee) due to increase in offgas activity and 7 x 7 fuel assembly cladding degradation.

+LAST IE SITE INSPECTION DATE: November 5 through December 4, 1979

+ INSPECTION REPORT NO: 79-26

SUMMARY

INSPECTION SUMMARY

+ Inspection on September 26-27; October 3-4, 22-23 and November 7-9, (79-20): Follow up to IE Bulletin No. 79-02, pipe support base plate designs using concrete expansion anchor bolts; review of test program and procedures; observation of testing anchor bolts; and review of quality records. The inspection involved 68 inspector hours onsite by one NRC inspector. No items of noncompliance were identified in the areas inspected.

+ Inspection on November 5 through December 4, (79-26): Routine, unannounced inspection of plant operations; physical protection - security organization, physical barriers, access control (identification, authorization, badging, search, and escorting), and communications; and, corporate management meeting. The inspection involved 53 inspector hours onsite by one NRC inspector and three manhours at the corporate office during a management meeting by three NRC personnel. No items of noncompliance were identified.

REPORTS RECEIVED FROM LICENSEE

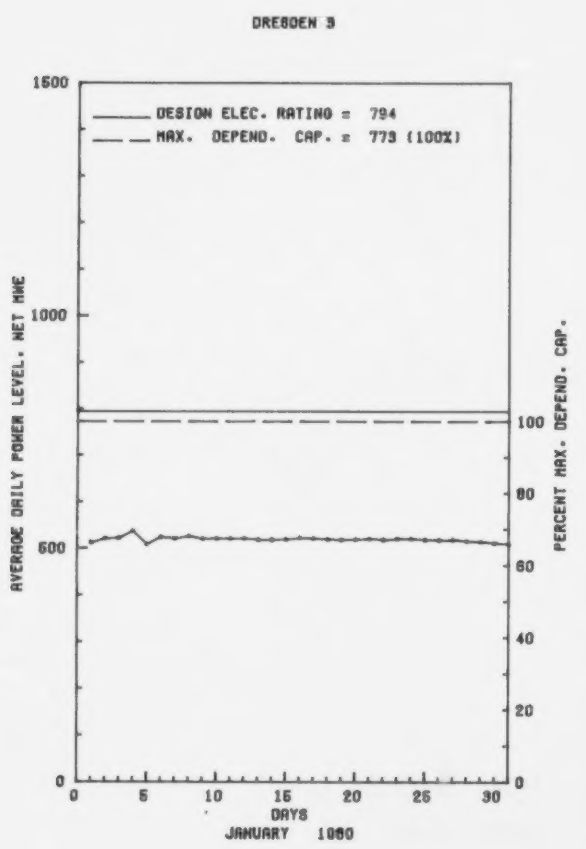
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-29/ 03L-0	11/01/79	11/28/79	30 Day LER	IRM Channel 11 tripped in excess of TS limits.
79-30/ 03L-0	11/10/79	12/06/79	30 Day LER	Failure of RPIS on CRD H-9.
79-31/ 03L-0	11/17/79	12/07/79	30 Day LER	MSIV 3-203-2D closure less than Tech Spec limits.
79-32/ 03L-0	11/20/79	12/10/79	30 Day LER	Pinhole leak found on 3B Reactor feed pump.
79-33/ 03L-0	12/10/79	01/07/80	30 Day LER	Failure of RBM channel 8.
79-34/ 03L-0	12/18/79	01/14/80	30 Day LER	One PSID dp lost between torus and drywell-lost while testing A03-161-21.
79-35/ 03L-0	12/17/79	01/11/80	30 Day LER	During normal operation control room operator observed no position indication for valve MO-3-1501-3A.

REVIEWED BY OMPA LOEB: DATE: 3/20/80

REVIEWED BY NRR DOR: RBBevan DATE: 3/21/80

DRESDEN 3

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-249
2. Utility Contact: S. A. Cieszkiewicz (815) 942-2920 x263
3. Licensed Thermal Power (MWt): 2527
4. Nameplate Rating (Gross MWe): 920 x 0.9 = 828
5. Design Electrical Rating (Net MWe): 794
6. Maximum Dependable Capacity (Gross MWe): 812
7. Maximum Dependable Capacity (Net MWe): 773
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): 550
10. Reasons for Restrictions, If Any: ADMIN. DERATING FOR AIR EJECTOR EVAL.

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	744.0	74,785.0
12. Hours Reactor Was Critical	744.0	744.0	56,509.4
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	744.0	744.0	54,068.6
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,275,133	1,275,133	103,546,801
17. Gross Electrical Energy (MWH)	407,966	407,966	33,837,640
18. Net Electrical Energy (MWH)	384,540	384,540	32,077,831
19. Unit Service Factor	100.0	100.0	72.3
20. Unit Availability Factor	100.0	100.0	72.3
21. Unit Capacity Factor (MDC Net)	66.9	66.9	55.5
22. Unit Capacity Factor (DER Net)	65.1	65.1	54.0
23. Unit Forced Outage Rate	.0	.0	16.3

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUEL, MARCH 17, 1980, APPROX 10 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	NONE								

SUMMARY

OPERATION WAS ROUTINE BELOW AN ADMINISTRATIVE DERATING TO 550 MWE FOR AIR EJECTOR EVALUATIONS THROUGHOUT THE REPORT PERIOD. NO FURTHER REDUCTIONS OR OUTAGES WERE REPORTED.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

NRC POWER RESTRICTIONS: NONE

DUANE ARNOLD

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Iowa/Linn 8 miles NW of Cedar Rapids
2.	Electric Reliability Council	N/A
3.	Type of Reactor	BWR
4.	Power Level (MWT)	1658
5.	Design Electrical Rating (MWE Net)	538
6.	Date of Initial Criticality	March 23, 1974
7.	Date of Commercial Operation	February 1, 1975
8.	Condenser Cooling Method	Cooling Towers
9.	Condenser Cooling Water Source	Cedar Rapids River Circulating Water System

Utility & Contractor Information

10.	Licensee	Iowa Electric Light and Power
11.	Corporate Address	IE Towers, P. O. Box 351 Cedar Rapids, Iowa 52406
12.	Corporate Contact	D. Arnold, President
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	GE
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Senior Inspector	R. Spessard
19.	IE Principal Inspector	G. Wright
20.	Licensing Project Manager	T. Kevern
21.	Docket Number	50-331
22.	License Number/Date of Issue	DPR-49 issued 2/22/74
23.	Public Document Room	Reference Service Cedar Rapids Public Library 428 Third Avenue, S.E. Cedar Rapids, Iowa 52401

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

```
    INSPECTION STATUS

    OTHER ITEMS OF INTEREST

    PLANT STATUS

+   Coastdown for refueling.

+LAST IE SITE INSPECTION DATE:  December 18-20, 1979

+        INSPECTION REPORT NO:  79-31

    SUMMARY

    INSPECTION SUMMARY

+   Inspection on December 18-20, (79-31):  Routine, unannounced inspection of
    Emergency Planning, including licensee internal audits; discussions with
    offsite support agencies; review of emergency facilities, equipment,
    instrumentation, and supplies specified in the Preparedness Plan and
    associated procedures; review of training of emergency personnel; review
    records of emergency exercises; and review of Environmental Technical
    Specification Violation 79-02.  The inspection involved 18 inspector hours
    onsite by one NRC inspector.  Of the seven areas inspected, no items of
    noncompliance or deviations were identified in six areas; one apparent item
    of noncompliance (excessive chlorine discharge) was identified in one area.
```

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-34/ 03L-0	11/21/79	12/21/79	30 Day LER	LPCI trip system "B" delay timer K-70B was found to have an out of spec. delay constant.
79-35/ 03L-0	11/21/79	12/21/79	30 Day LER	"A" river water supply pump motor 1P-117C to be vibrating excessively.
79-36/ 03L-0	10/29/79	01/07/80	30 Day LER	RCIC drain pot level control valve CV-2409 packing ruptured causing steam to be discharged to the room.
79-37/ 01T-0	12/25/79	01/07/80	14 Day LER	Loss of both channels of the core spray sparger break detection alarm system.
79-38/ 03L-0	12/21/79	01/17/80	30 Day LER	Operation pen failed.

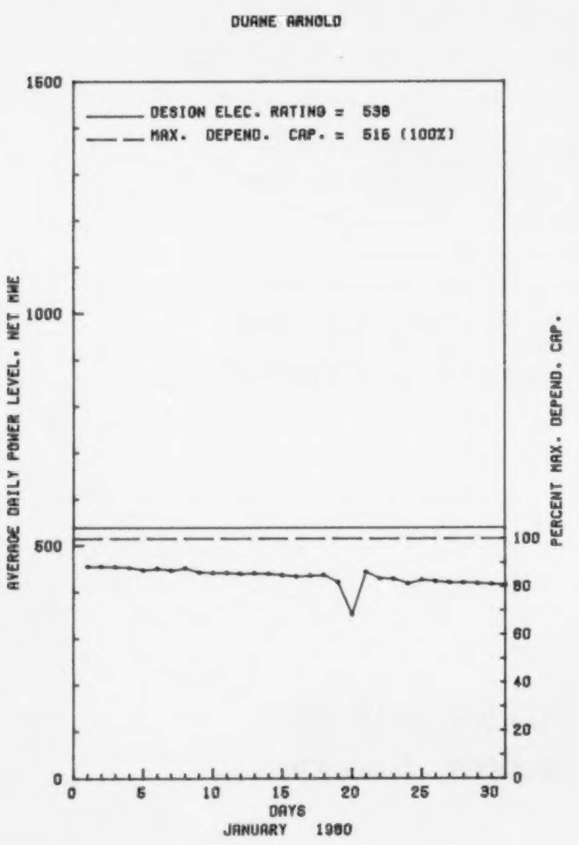
==

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 3-19-80

REVIEWED BY NRR DOR: Thomas A. [illegible] **DATE:** 3/20/80

DUANE ARNOLD

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-331
2. Utility Contact: J. Van Sickel (319) 851-5611
3. Licensed Thermal Power (MWt): 1658
4. Nameplate Rating (Gross MWe): 663 x 0.9 = 597
5. Design Electrical Rating (Net MWe): 538
6. Maximum Dependable Capacity (Gross MWe): 545
7. Maximum Dependable Capacity (Net MWe): 515
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	744.0	43,824.0
12. Hours Reactor Was Critical	744.0	744.0	31,308.1
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	744.0	744.0	30,563.1
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	976,200	976,200	38,105,760
17. Gross Electrical Energy (MWH)	343,643	343,643	12,768,009
18. Net Electrical Energy (MWH)	322,176	322,176	11,937,240
19. Unit Service Factor	100.0	100.0	69.7
20. Unit Availability Factor	100.0	100.0	69.7
21. Unit Capacity Factor (MDC Net)	84.1	84.1	52.9
22. Unit Capacity Factor (DER Net)	80.5	80.5	50.6
23. Unit Forced Outage Rate	.0	.0	20.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
REFUELING, FEBRUARY 9, 1980, 10 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	NONE								

SUMMARY

OPERATION WAS ROUTINE IN BASE LOADED COASTDOWN OPERATION THROUGHOUT THE REPORT PERIOD. POWER WAS REDUCED BRIEFLY ON 01/20/80 TO PERFORM CONTROL ROD SCRAM TIME TESTING.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

FARLEY 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Alabama/Houston 24 miles SE of Dothan
2.	Electric Reliability Council	SERC
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2652
5.	Design Electrical Rating (MWE Net)	829
6.	Date of Initial Criticality	August 9, 1977
7.	Date of Commercial Operation	December 1, 1977
8.	Condenser Cooling Method	Once Through With Forced Draft Cooling Towers
9.	Condenser Cooling Water Source	Chattahoochee River

Utility & Contractor Information

10.	Licensee	Alabama Power Company
11.	Corporate Address	600 North 18th Street Birmingham, Alabama
12.	Corporate Contact	F. L. Clayton, Jr. Senior Vice President (205-323-5341)
13.	Architect/Engineer	Bechtel/SSI
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Daniel

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. Lewis
18.	IE Senior Inspector	R. D. Martin
19.	IE Principal Inspector	W. H. Bradford
20.	Licensing Project Manager	E. A. Reeves
21.	Docket Number	50-348
22.	License Number/Date of Issue	NPF-2, 6/25/77
23.	Public Document Room	G. S. Houston Memorial Library 212 W. Berdeshaw Street Dothan, Alabama 36301

INSPECTION STATUS

ENFORCEMENT STATUS

FAILURE TO CONDUCT REQUIRED TEST OF ALARM SYSTEM.
(7941 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

ROUTINE OPERATIONS

LAST IE SITE INSPECTION DATE: JANUARY 1-31, 1980

INSPECTION REPORT NO: 50-348/80-01

SUMMARY

INSPECTION SUMMARY

INSPECTION DECEMBER 12-14 (79-39): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 18 INSPECTOR-HOURS ON SITE IN THE AREAS OF REVIEW OF PREOPERATIONAL TESTING ACTIVITIES ON UNIT 2 AND GENERAL SITE FAMILIARIZATION. OF THE AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION DECEMBER 10-14 (79-41): INCLUDED REVIEW OF SECURITY PLAN, SECURITY PROGRAM AUDIT, SECURITY ORGANIZATION, PROTECTED AND VITAL AREA PHYSICAL BARRIERS, LIGHTING, ACCESS CONTROLS, DETECTION AIDS, ASSESSMENT AIDS, COMMUNICATIONS, TESTING AND MAINTENANCE, COMPENSATORY MEASURES, RESPONSE, LOCKS/KEYS, TESTING/MAINTENANCE AND LICENSEE ACTION ON PREVIOUS INSPECTION FINDINGS. THE INSPECTION INVOLVED 36 INSPECTOR-HOURS ON SITE BY 1 NRC INSPECTOR. THE INSPECTION WAS BEGUN DURING AN OFFSHIFT PERIOD; 6 INSPECTION HOURS WERE ACCOMPLISHED DURING OFFSHIFT PERIODS. THE LICENSEE WAS FOUND TO BE IN COMPLIANCE WITH NRC REQUIREMENTS WITHIN THE AREAS EXAMINED DURING THE INSPECTION EXCEPT FOR THE FOLLOWING ITEMS: TESTING AND MAINTENANCE - ONE INTRUSION ALARM TEST WAS NOT CONDUCTED DURING THE PERIOD EXAMINED (DEFICIENCY 79-41-01).

INSPECTION DECEMBER 5-7 (79-43): THIS ROUTINE UNANNOUNCED INSPECTION INVOLVED 16 INSPECTOR-HOURS ON SITE IN THE AREAS OF ASME CERTIFICATIONS, QA MECHANICAL AUDITS, CDR'S, LER'S, UNRESOLVED ITEMS AND INSPECTOR FOLLOWUP ITEMS. OF THE SEVEN AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION DECEMBER 19-21 (79-44): MATERIAL CONTROL AND ACCOUNTING. THE INSPECTION INVOLVED 18 INSPECTOR-HOURS ON SITE BY ONE NRC INSPECTOR AND WAS BEGUN DURING THE REGULAR HOURS. THE LICENSEE WAS FOUND TO BE IN COMPLIANCE WITH NRC REQUIREMENTS IN THE ONE AREA EXAMINED.

INSPECTION DECEMBER 3-29 (79-45): THIS ROUTINE, INSPECTION INVOLVED 96 INSPECTOR-HOURS ON SITE IN THE AREAS OF PLANT OPERATIONS, UNIT NO. 2 PREOPERATIONAL STARTUP TEST PROGRAM, PHYSICAL SECURITY AND OPEN ITEMS. OF THE FOUR AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION DECEMBER 27 (79-46): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED THREE INSPECTOR-HOURS ON SITE IN THE AREAS AND ACTIVITIES RELATIVE TO IE BULLETIN 79-17 (PIPE CRACKS IN STAGNANT BORATED WATER SYSTEMS AT PWR PLANTS). NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

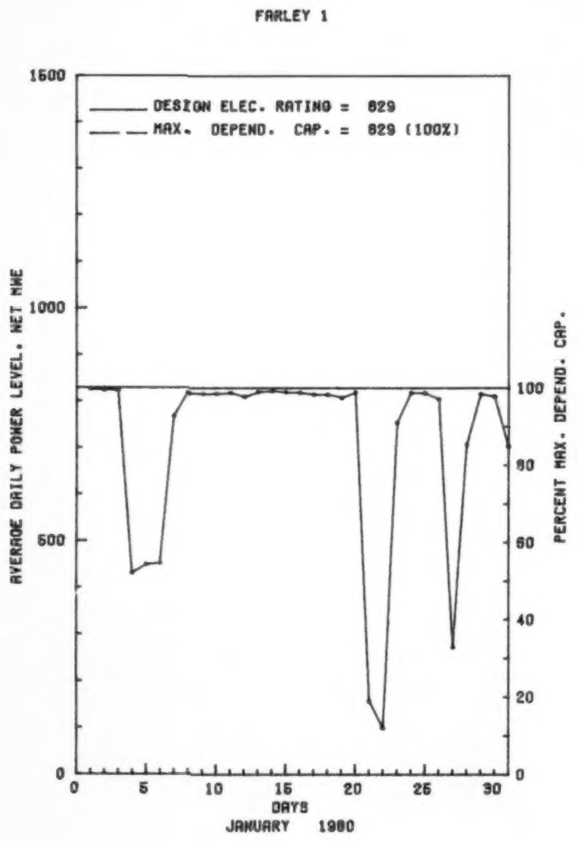
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-050/ 03L-0	11/07/79	12/03/79	30 DAY LER	PRESSURIZER PRESS TRANSMITTER PT445 BISTABLES WERE NOT TRIPPED WHEN TRANSMITTER WAS ISOLATED TECH SPEC 6.9.1.9.C VIOLATED.
79-051/ 03L-0	11/03/79	12/03/79	30 DAY LER	TURBINE DRIVEN AUXFEEDWATER PUMP (TDAFWP) TRIPPED ON OVERSPEED DUE TO MISPOSITIONED MANUAL SPEED CONTROL. REPORTABLE PER TECH SPEC 3.7.1.2.
79-052/ 03L-0	11/06/79	12/03/79	30 DAY LER	REFUELING WTR STORAGE TANK WAS 14721 GAS LOW OUT OF SPEC LEVEL XMITTERS HAD DRIFTED LOW. ONE RECALIBRATED N RETURNED TO USE, ONE STILL WAITING PARTS.
79-053/ 03L-0	11/01/79	11/30/79	30 DAY LER	ROD K14 IN CONTROL BANK A WAS NOT ABLE TO MOVE IN OR OUT BUT WAS TRIPABLE. CAUSE - OPEN IN CABLE TO LIFT COIL.
79-054/ 01T-0	11/19/79	12/03/79	14 DAY LER	WESTINGHOUSE NEG RATE TRIP MAY NOT OCCUR FOR SOME SINGLE MVLT ROD DROPS, CALC USED NONCONSERVATIVE ASSUMPTIONS, DNBR LIMIT COULD BE VIOLATED WHEN OVER 90.
79-055/ 03L-0	11/08/79	12/07/79	30 DAY LER	MAKEUP TO RCS DURING COOLDOWN RESULTED IN BAT VOLUME BELOW TECH SPEC 3.1.2.8 A.1 BY 442GL.
79-056/ 03L-0	11/08/79	12/07/79	30 DAY LER	20 FIRE BARRIERS WERE FOUND TO NOT BE FUNCTIONAL.
79-059 03L-0	11/30/79	12/31/79	30 DAY LER	VALVE FOUND SHUT IN BORIC ACID FLOW PATH.
79-060/ 03L-0	12/02/79	12/28/79	30 DAY LER	CONTAINMENT PRESSURE CHANNEL FAILED DURING STARTUP.
79-062/ 03L-0	12/02/79	01/03/80	30 DAY LER	INDICATED AXIAL FLUX DIFFERENCE NOT WITHIN TARGET BAND FOR 11 MINUTES DURING A LOAD REDUCTION DUE TO FAILED VOLTAGE/FREQ.
79-066/ 03L-0	12/20/79	01/16/80	30 DAY LER	CONTAINMENT PRESSURE CHANNEL PT950 WAS FOUND INOPERABLE DUE FAULTY CONNECTION IN "B" SOLID STATE PROTECTION SYSTEM

REVIEWED BY OMPA LOEB: T. C. Cintula DATE: 3·19·80

REVIEWED BY NRR DOR: Ed A. Reeves DATE: 3-20-80

FARLEY 1

AVERAGE POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-348
2. Utility Contact: W. G. Hairston,III (205) 899-5156
3. Licensed Thermal Power (MWt): 2652
4. Nameplate Rating (Gross MWe): 1045 x 0.85 = 888
5. Design Electrical Rating (Net MWe): 829
6. Maximum Dependable Capacity (Gross MWe): 860
7. Maximum Dependable Capacity (Net MWe): 829
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	744.0	19,008.0
12. Hours Reactor Was Critical	705.4	705.4	11,640.9
13. Reactor Reserve Shutdown Hours	38.6	38.6	1,443.9
14. Hours Generator On-Line	696.2	696.2	11,286.7
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,721,418	1,721,418	28,319,433
17. Gross Electrical Energy (MWH)	557,470	557,470	9,081,508
18. Net Electrical Energy (MWH)	526,908	526,908	8,586,324
19. Unit Service Factor	93.6	93.6	59.4
20. Unit Availability Factor	93.6	93.6	59.4
21. Unit Capacity Factor (MDC Net)	85.4	85.4	54.5
22. Unit Capacity Factor (DER Net)	85.4	85.4	54.5
23. Unit Forced Outage Rate	6.2	6.2	7.3

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 02/12/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
001	01/04/80	F	2.0	B	5		HH	PUMPXX	UNIT WAS REDUCED TO 35% REACTOR POWER IN ORDER TO REMOVE ONE OF THE TWO OPERABLE CONDENSATE PUMPS FROM SERVICE FOR MAINTENANCE.
002	01/04/80	F	73.0	B	5		HH	PUMPXX	UNIT WAS LIMITED TO 57% REACTOR POWER WITH ONLY ONE OPERABLE CONDENSATE PUMP AVAILABLE FOR SERVICE.
003	01/12/80	S	2.7	B	5		HA	VALVEX	UNIT WAS REDUCED TO 90% REACTOR POWER TO PERFORM TURBINE GOVERNOR VALVE SURVEILLANCE TESTING.
004	01/19/80	S	2.4	B	5		HA	VALVEX	UNIT WAS REDUCED TO 90% REACTOR POWER TO PERFORM TURBINE GOVERNOR VALVE SURVEILLANCE TESTING.
005	01/21/80	F	26.5	B	1		CI	ZZZZZZ	UNIT WAS TAKEN OFF-LINE TO INVESTIGATE IDENTIFIED REACTOR COOLANT SYSTEM LEAKAGE. DRAIN VALVE LEAKAGE WAS LOCATED AND TERMINATED.
006	01/22/80	F	7.4	B	3		HH	CKTBRK	UNIT TRIPPED DUE TO LOSS OF 1B SGFP ON A REVERSE THRUST BEARING TRIP CAUSED WHILE TROUBLESHOOTING BREAKER TRIP INDICATION FOR 1B SGFP TURNING GEAR MOTOR.
007	01/26/80	S	2.3	B	5		HA	VALVEX	UNIT WAS REDUCED TO 90% REACTOR POWER TO PERFORM TURBINE GOVERNOR VALVE SURVEILLANCE TESTING.
008	01/27/80	F	12.0	A	3		HH	PIPEXX	UNIT TRIPPED DUE TO LOSS OF BOTH SGFP'S ON LOW SUCTION PRESSURE. THE INSTRUMENT AIR LINE TO SGFP 1A MINI-FLOW VALVES TO FAIL OPEN. THE FAILED SECTION OF LINE WAS REPLACED AND THE UNIT RETURNED ON LINE.
009	01/28/80	F	12.6	A	5		AA	BLOWER	UNIT WAS REDUCED TO 65% REACTOR POWER, THEN LIMITED TO 85% REACTOR POWER TO MAINTAIN THE ISOPHASE BUS COOLING SYSTEM BELOW 170 DEG. F. DUE TO THE LOSS OF AN ISOPHASE BUS DUCT COOLING FAN.
010	01/30/80	S	1.0	B	5		HA	VALVEX	UNIT WAS REDUCED TO 90% REACTOR POWER TO SWITCH FROM MANUAL TO AUTO. CONTROL OF THE TURBINE GOVERNOR VALVES FOLLOWING STEAM GENERATOR MOISTURE CARRYOVER TESTING.
011	01/31/80	S	2.0	F	1		ZZ	ZZZZZZ	UNIT WAS SHUTDOWN TO PERFORM NUREG 0578 RELATED DESIGN CHANGES.

SUMMARY

THE UNIT EXPERIENCED SEVERAL POWER REDUCTIONS DURING THE REPORT PERIOD, FOR MAINTENANCE ON CONDENSATE PUMPS, TURBINE GOVERNOR VALVE SURVEILLANCE TESTING, AND DUE TO LOSS OF AN ISOPHASE BUS DUCT COOLING FAN. OUTAGES OCCURRED FOR INVESTIGATION OF REACTOR COOLANT SYSTEM LEAKAGE, WITH TRIPS WHILE TROUBLESHOOTING A BREAKER TRIP INDICATION AND FROM LOSS OF BOTH STEAM GENERATOR FEED PUMPS ON LOW SUCTION PRESSURE, AND TO PERFORM NUREG-0578 RELATED DESIGN CHANGES.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

FITZPATRICK

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	New York/Oswego 8 miles NE of Oswego
2.	Electric Reliability Council	NPCC
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2436
5.	Design Electrical Rating (MWE Net)	821
6.	Date of Initial Criticality	November 17, 1974
7.	Date of Commercial Operation	July 28, 1975
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Lake Ontario

Utility & Contractor Information

10.	Licensee	Power Authority of the State of New York
11.	Corporate Address	10 Columbus Circle New York, New York 10019
12.	Corporate Contact	P. Lyon, Manager of Nuclear Operations
13.	Architect/Engineer	Stone & Webster
14.	Nuclear Steam Supply System	GE
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner (X246)
18.	IE Senior Inspector	E. McCabe (X251)
19.	IE Principal Inspector	T. F. Stetka (X276)
20.	Licensing Project Manager	P. Polk (X7872)
21.	Docket Number	50-333
22.	License Number/Date of Issue	DPR-59, 10/17/74
23.	Public Document Room	State University College of Oswego Penfield Library - Documents Oswego, NY 13126 (315)341-2323

INSPECTION STATUS

ENFORCEMENT STATUS

10 CFR 50.59 STATES, IN PART, "THE LICENSEE SHALL MAINTAIN RECORDS OF CHANGES IN THE FACILITY.... THESE RECORDS SHALL INCLUDE A WRITTEN SAFETY EVALUATION.... THE LICENSEE SHALL FURNISH TO THE APPROPRIATE NRC REGIONAL OFFICE...ANNUALLY...A BRIEF DESCRIPTION OF...CHANGES...INCLUDING A SUMMARY OF THE SAFETY EVALUATION OF EACH." CONTRARY TO THE ABOVE, THE LICENSEE DID NOT SUBMIT A REPORT OF CHANGES TO THE FACILITY INCLUDING A BRIEF SUMMARY OF THEIR SAFETY EVALUTIONS FOR THE MODIFICATIONS COMPLETED DURING 1978.
(7913 DEFICIENCY)

10 CFR 50, APPENDIX B, CRITERION V, INSTRUCTION, PROCEDURES, AND DRAWINGS, STATES, IN PART, "ACTIVITIES AFFECTING QUALITY SHALL BE PRESCRIBED BY DOCUMENTED INSTRUCTIONS, PROCEDURES, AND DRAWINGS, OF A TYPE APPROPRIATE TO THE CIRCUMSTANCES AND SHALL BE ACCOMPLISHED IN ACCORDANCE WITH THESE INSTRUCTIONS, PROCEDURES, AND DRAWINGS." THE QUALITY ASSURANCE PROGRAM MANUAL, SECTION 5, STATES, IN PART, "QUALITY AFFECTING ACTIVITIES SHALL BE BASED ON...DRAWINGS...AS APPROPRIATE." CONTRARY TO THE ABOVE, THE DRAWINGS USED TO CONDUCT PLANT OPERATIONS (FMS AND OPS) WERE INCONSISTENT. THE FOLLOWING EXAMPLES ARE SEVERAL OF THE DISCREPANCIES NOTED BY THE INSPECTOR BETWEEN THE OPS AND FMS: FM-22A SHOWS FOUR CONTAINMENT ISOLATION VALVES ON INSTRUMENTATION LINES WHICH ARE NOT SHOWN ON OP 19/19-1 EVEN THOUGH BOTH DRAWINGS DEPICT THE SAME PIPING. FM-16A SHOWS AN AIR OPERATED VALVE BETWEEN THE OFFGAS FILTERS AND THE STACK BUT OP 24A-1/24-1 DOES NOT, EVEN THOUGH BOTH DRAWINGS DEPICT THE SAME PIPING. FM-43A SHOWS DAMPER D-3 THAT IS NOT SHOWN ON OP-20/20-1 EVEN THOUGH THE DRAWINGS DEPICT THE SAME SYSTEM.
(7913 DEFICIENCY)

10 CFR 50 APPENDIX B, CRITERION XVI, STATES THAT MEASURES SHALL BE ESTABLISHED TO ASSURE THAT CONDITIONS ADVERSE TO QUALITY ARE PROMPTLY CORRECTED. THE FSAR, APPENDIX D, SECTION D.2.2.0 STATES THAT THE QA PROGRAM COMPLIES WITH WASH 1283, DATED MAY 24, 1974. WASH 1283 ENDORSES ANSI N45.2.12-1974. ANSI N45.2.12-1974 STATES THAT THE AUDITED ORGANIZATION SHALL RESPOND TO AN AUDIT REPORT WITHIN 30 DAYS OF THE RECEIPT OF THE RESULTS OF THE AUDIT. THE RESPONSE SHALL CLEARLY STATE THE CORRECTIVE ACTION TAKEN OR A SCHEDULED DATE BY WHICH CORRECTIVE ACTION WILL BE COMPLETED. CONTRARY TO THE ABOVE, THE ENGINEERING DEPARTMENT HAD NOT RESPONDED TO AUDIT REPORT NO. 1 OF THE IMPLEMENTATION OF ENGINEERING DEPARTMENT PROCEDURES AS OF SEPTEMBER 19, 1979. THE RESULTS OF AUDIT REPORT NO. 1 WERE FORWARDED TO THE ENGINEERING DEPARTMENT ON OCTOBER 5, 1978.
(7913 INFRACTION)

THE J. A. FITZPATRICK TECHNICAL SPECIFICATIONS, SECTION 6.1 STATES THAT THE RESIDENT MANAGER REPORTS TO THE GENERAL MANAGER AND CHIEF ENGINEER (A SINGLE POSITION). THE INDIAN POINT NO. 3 TECHNICAL SPECIFICATIONS, SECTION 6.2.1 STATES THAT THE ORGANIZATION SHALL BE AS SHOWN ON FIGURE 6.2.1 WHICH SHOWS THE RESIDENT MANAGER REPORTING TO THE GENERAL MANAGER AND CHIEF ENGINEER. CONTRARY TO THE ABOVE, THE RESIDENT MANAGERS AT J. A. FITZPATRICK AND INDIAN POINT NO. 3 WERE REPORTING TO THE DIRECTOR OF POWER OPERATIONS, WHO IS ADMINISTRATIVELY, AT A LOWER LEVEL THAN THE GENERAL MANAGER AND CHIEF ENGINEER. A FORMAL REQUEST FOR A CHANGE IN TECHNICAL SPECIFICATIONS WAS NOT SUBMITTED FOR FITZPATRICK AND NRC APPROVAL WAS NOT GRANTED FOR EITHER FACILITY.
(7913 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

+ An approximate 6 inch axial pipe crack has developed on the suction line to condensate booster pump B. A temporary fix has been made. Reactor thermal limits reduced to 2 percent less than rated to prevent exceeding CMFLPD limit.

FACILITY ITEMS (PLANS AND PROCEDURES):

+ Refueling outage scheduled for April, 1980.

MANAGERIAL ITEMS:

None

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-033/03X	10/08/79	11/09/79	30 Day LER	MFLPD exceeded TS limit during normal operations
79-093/03X	10/21/79	11/28/79	30 Day LER	CMFLPD was found to be out of specification during routine check
79-103/01T	12/05/79	12/17/79	Other	Analysis determined that one support on the B RHR steam condensing mode heat exchanger was inoperative
79-106/03L	11/18/79	12/17/79	30 Day LER	MSIV 29-ADV-80A limit switch does not provide half-scram signal on 10 percent closure of valve
79-107/03L	12/03/79	12/26/79	30 Day LER	Primary containment hydrogen analyzer went out of calibration and containment atmosphere was sampled using portable equipment
79-108/03L	12/06/79	01/03/80	30 Day LER	RCIC pump made inoperable to check pump bearings
79-109/03L	12/13/79	01/08/80	30 Day LER	Drywell and suppression pool temperature instrument went out of calibration due to a failed printed circuit board
79-110/03L	12/14/79	01/08/80	30 Day LER	HPCI system steam supply isolation valve closed due to spurious isolation signal
79-111/03L	12/14/79	01/10/80	30 Day LER	HPCI/RCIC steam line high flow instrument found out of calibration during testing
79-112/03L	12/16/79	01/11/80	30 Day LER	Fire doors DG-272-2 and DG-272-3 did not close during testing due to door misalignment
79-113/03L	12/21/79	01/15/80	30 Day LER	Core spray valve 14-MOV-11B failed to reopen during testing
79-114/04T	12/21/79	01/03/80	14 Day LER	Co-60 sample activity was greater than ten times the control sample activity when measured in accordance with ETS requirements
79-115/03L	12/21/79	01/15/80	30 Day LER	Stack gas radiation monitor system inoperable due to plugged sample line
79-116/03L	12/27/79	01/16/80	30 Day LER	SRMS failed at various times during reactor operation and while shutdown
79-117/03L	12/27/79	01/17/80	30 Day LER	Setpoint drift on APRM D scram trip point

PLANT STATUS:

+ The plant operated routinely during the month of January. Power was reduced to perform a control rod change as the unit is approaching end of core life.

+ LAST IE SITE INSPECTION DATE: 2/15/80

+ INSPECTION REPORT NO: 50-333/80-01

SUMMARY

INSPECTION SUMMARY

50-333/79-17 - Dec 11-14: Routine, unannounced inspection of: plant operations, Licensee Event Reports (LERs); Personnel Qualifications; IE Bulletin and Circular responses; and licensee action on previous inspection reports. The inspectors participated in an NRC Team review of the Small Break Loss of Coolant Accident Procedure Implementation. The report also documents in-office review of LERs. The inspection involved 67 hours onsite by two region-based inspectors. No items of noncompliance were identified.

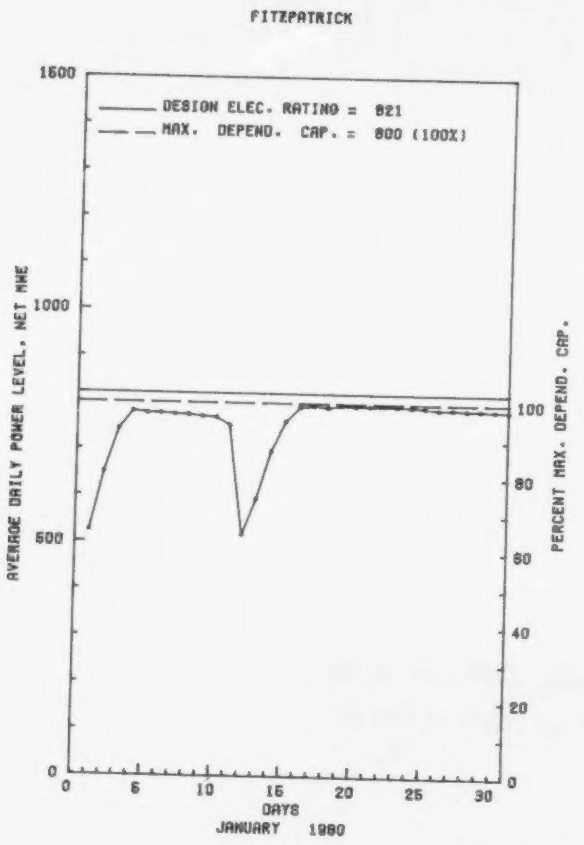
+ 50-333/79-18 - Dec 12-14: Special, unannounced inspection of the licensee's response to IE Bulletin 79-19 and of the licensee's radiation protection program. Areas inspected included document control, procedures, training, audits, facility tours, and waste solidification. The inspection involved 17 hours onsite by one NRC region-based inspector. One item of noncompliance was identified (Failure to follow procedures).

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 3-19-80

REVIEWED BY NRR DOR: Philip J. Polk **DATE:** 3-21-80

FITZPATRICK

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

ITEM 21 CALCULATED BY USING A WEIGHTED AVERAGE

1. Docket Number: 50-333
2. Utility Contact: E. Zufelt (315) 342-3840
3. Licensed Thermal Power (MWt): 2436
4. Nameplate Rating (Gross MWe): 981 x 0.9 = 883
5. Design Electrical Rating (Net MWe): 821
6. Maximum Dependable Capacity (Gross MWe): 830
7. Maximum Dependable Capacity (Net MWe): 800
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	744.0	35,577.0
12. Hours Reactor Was Critical	744.0	744.0	27,374.2
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	744.0	744.0	26,482.0
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,674,240	1,674,240	52,862,434
17. Gross Electrical Energy (MWH)	577,890	577,890	17,971,590
18. Net Electrical Energy (MWH)	559,935	559,935	17,350,510
19. Unit Service Factor	100.0	100.0	66.9
20. Unit Availability Factor	100.0	100.0	66.9
21. Unit Capacity Factor (MDC Net)	94.1	94.1	60.4
22. Unit Capacity Factor (DER Net)	91.7	91.7	53.4
23. Unit Forced Outage Rate	.0	.0	20.6

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): SNUBBER INSPECT & REFUELING - APRIL 1980

25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
1	01/12/80	S	.0	H	5				REDUCED POWER TO PERFORM CONTROL ROD SEQUENCE EXCHANGE.

SUMMARY

OPERATION WAS AT NEAR FULL POWER DURING THE REPORT PERIOD, WITH A LOAD REDUCTION ON 01/12/80 FOR CONTROL ROD SEQUENCE EXCHANGE.

LEGEND

[1] Type:
F-Forced
S-Scheduled

[2] Reason:
A-Equipment Failure
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error
H-Other

[3] Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Continuations
5-Load Reduction
9-Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

FORT CALHOUN

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Nebraska/Washington 19 miles N of Omaha
2.	Electric Reliability Council	MARCA
3.	Type of Reactor	PWR
4.	Power Level (MWT)	1420
5.	Design Electrical Rating (MWE Net)	457
6.	Date of Initial Criticality	August 6, 1973
7.	Date of Commercial Operation	June 20, 1974
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Missouri River

Utility & Contractor Information

10.	Licensee	W. C. Jones Division Manager – Production Operations Omaha Public Power District
11.	Corporate Address	1623 Harney Street Omaha, Nebraska 68102
12.	Corporate Contact	T. E. Short Assistant General Manager
13.	Architect/Engineer	Gibbs & Hill, Inc.
14.	Nuclear Steam Supply System	Combustion Engineering
15.	Constructor	Gibbs & Hill, Durham & Richardson, Inc.

Regulatory Information

16.	IE Region responsible	IV
17.	IE Branch Chief	G. L. Madsen
18.	IE Senior Inspector	T. F. Westerman
19.	IE Principal Inspector	D. Kelley
20.	Licensing Project Manager	P. B. Erickson
21.	Docket Number	50-285
22.	License Number/Date of Issue	DPR-40, 5/24/73
23.	Public Document Room	W. Dale Clark Library 215 S. 15th Street Omaha, Nebraska 68102

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

```
INSPECTION STATUS

OTHER ITEMS OF INTEREST

     SYSTEMS AND COMPONENT PROBLEMS:

     (1)  Concrete shield wall at equipment hatch not installed per
          FSAR figure 11.2.2 (76-16)

     FACILITY ITEMS (PLANS AND PROCEDURES):


     MANAGERIAL ITEMS:

     (1)  Safety Audit and Review Committee review of Facility
          License Changes. (76-16)


     PLANT STATUS:

     No significant information

LAST IE SITE INSPECTION DATE:  December 10 - 13, 1979

INSPECTION REPORT NO:  50-285/79-20

 SUMMARY

 INSPECTION SUMMARY

 NO INSPECTION CONDUCTED
```

REPORTS RECEIVED FROM LICENSEE

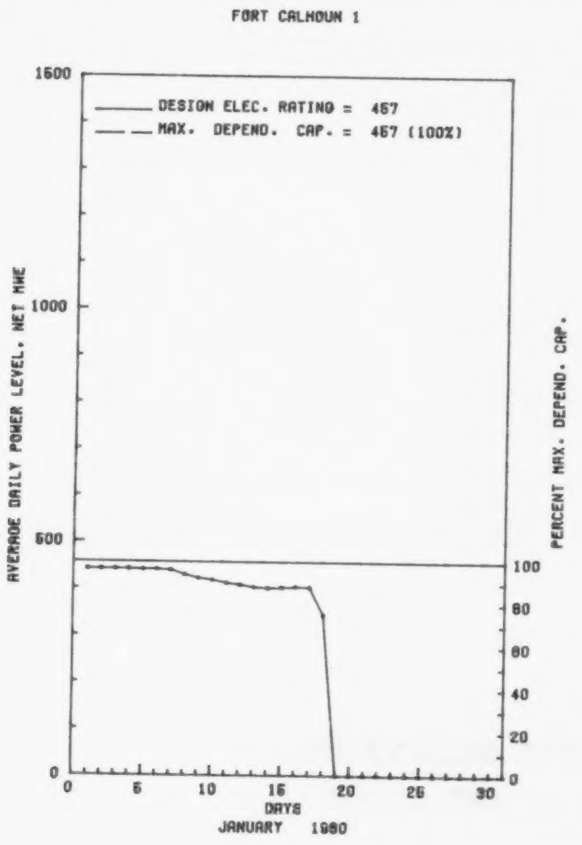
```
==========================================================================================
 NUMBER   DATE OF   DATE OF                           SUBJECT
           EVENT     REPORT
==========================================================================================
 79-22    12/18/79  12/20/79    RADIATION MONITOR TRIP RELAY FAILURE - ONE CHANNEL
 03L-0
```

REVIEWED BY OMPA LOEB: MR Beebe **DATE:** 03/19/80

REVIEWED BY NRR DOR: P B Erickson **DATE:** 3/20/80

FORT CALHOUN

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

ITEM 21 CALCULATED BY USING A WEIGHTED AVERAGF

1. Docket Number: 50-285
2. Utility Contact: B. J. Hickle (402) 536-4413
3. Licensed Thermal Power (MWt): 1420
4. Nameplate Rating (Gross MWe): 591 x 0.85 = 502
5. Design Electrical Rating (Net MWe): 457
6. Maximum Dependable Capacity (Gross MWe): 481
7. Maximum Dependable Capacity (Net MWe): 457
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	744.0	55,581.0
12. Hours Reactor Was Critical	432.9	432.9	44,834.5
13. Reactor Reserve Shutdown Hours	.0	.0	1,309.5
14. Hours Generator On-Line	430.5	430.5	43,881.6
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	574,731	574,731	53,839,758
17. Gross Electrical Energy (MWH)	190,604	190,604	17,850,834
18. Net Electrical Energy (MWH)	180,935	180,935	16,868,452
19. Unit Service Factor	57.9	57.9	78.8
20. Unit Availability Factor	57.9	57.9	78.8
21. Unit Capacity Factor (MDC Net)	53.2	53.2	66.8
22. Unit Capacity Factor (DER Net)	53.2	53.2	66.3
23. Unit Forced Outage Rate	.0	.0	4.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 03/15/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-01	01/18/80	S	313.5	C	1				REACTOR SHUTDOWN FOR ANNUAL REFUELING AND MAINTENANCE OUTAGE.

SUMMARY

OPERATION WAS ROUTINE IN AN END-OF-CYCLE COASTDOWN UNTIL 01/18/80, WHEN THE UNIT WAS TAKEN OFF-LINE FOR REFUELING.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

FORT ST. VRAIN

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Colorado/Weld 35 miles N of Denver (near Platteville)
2.	Electric Reliability Council	WSCC
3.	Type of Reactor	HTGR
4.	Power Level (MWT)	842
5.	Design Electrical Rating (MWE Net)	330
6.	Date of Initial Criticality	January 31, 1974
7.	Date of Commercial Operation	July 1, 1979
8.	Condenser Cooling Method	Towers
9.	Condenser Cooling Water Source	South Platte River/Shallow Wells

Utility & Contractor Information

10.	Licensee	Public Service Company of Colorado
11.	Corporate Address	P. O. Box 840 Denver, Colorado 80201
12.	Corporate Contact	C. K. Millen Senior Vice President
13.	Architect/Engineer	Sargent & Lundy
14.	Nuclear Steam Supply System	General Atomic Company
15.	Constructor	General Atomic Company

Regulatory Information

16.	IE Region responsible	IV
17.	IE Branch Chief	G. L. Madsen
18.	IE Senior Inspector	T. F. Westerman
19.	IE Resident Inspector	M. W. Dickerson
20.	Licensing Project Manager	G. Kuzmycz
21.	Docket Number	50-267
22.	License Number/Date of Issue	DPR-34, 12/21/73
23.	Public Document Room	Greeley Public Library City Complex Building Greeley, Colorado 80631

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO CRITERION V APPENDIX B AND ADM-30 COMBUSTIBLE ITEMS WERE NOT CONTROLLED AS REQUIRED & MAINTENANCE AND REPAIR PROCEDURES WERE NOT IN ACCORDANCE WITH ADM-12 REQUIREMENTS.
(7922 INFRACTION)

CONTRARY TO T/S 7.4.A REQUIREMENTS NO PROCEDURE HAD BEEN ESTABLISHED FOR THE CONTROL OF THE REMOVAL OF THE REACTOR FORCED COOLING.
(7922 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

Areas related to potential equipment modification, which are under review and discussion with the NRC include: helium circulator auxiliaries; accident analysis relating to outlet temperature dispersion and orifice settings; hangers for 2" and smaller piping; and plant oscillations.

FACILITY ITEMS (PLANS AND PROCEDURES):

Implementation of 10 CFR 73.55

MANAGERIAL ITEMS:

PLANT STATUS:

THE REACTOR WAS SHUT DOWN ON 1/28/80 FOR THE REPLACEMENT OF 1B HELIUM CIRCULATOR WHICH DEVELOPED A LEAK IN THE STATIC SEAL ACTUATING MECHANISM. THE FACILITY IS EXPECTED TO BE SHUT DOWN FOR APPROXIMATELY 60 DAYS.

NRC POWER RESTRICTIONS: Restricted to 70% of Licensed Thermal Power.

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
79-60 03L-0	11/28/79	12/20/79	THREE BEARING H2O PRESSURE DIFFERENTIAL SWITCHES WITH SETPOINTS LESS CONSERVATIVE THAN REQUIRED BY T/S-LCO 4.4-1
79-61 03L-0	11/20/79	12/20/79	PH & TDS SAMPLES NOT TAKEN. CHLORINE RESIDUAL DISCHARGE GREATER THAN LCO NR L1 LIMIT
79-62 03L-0	12/1/79	12/28/79	TWO MOISTURE MONITOR FLOW TRANSMITTERS OUT OF TOLERANCE LCO 4.4-1
79-63	CANCELLED		
79-64 03L-0	12/5/79	1/2/80	THREE SNUBBERS DETERMINED TO BE INOPERABLE. LCO 4.3.10
80-01 01T-0	1/8/80	1/22/80	HELIUM CIRCULATOR 1B STATIC SEAL LEAK

LAST IE SITE INSPECTION DATE: December 1 - 28, 1979

INSPECTION REPORT NO: 50-267/79-22

SUMMARY

INSPECTION SUMMARY

INSPECTION ON NOVEMBER 14-16, 1979, AND NOVEMBER 28-30, 1979 (79-19): ROUTINE, ANNOUNCED INSPECTION OF LICENSEE ACTIONS TAKEN IN RESPONSE TO IE BULLETIN 79-02, REVISION 2. NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION ON DECEMBER 3-7, 1979 (79-20): ROUTINE UNANNOUNCED INSPECTION INCLUDED A REVIEW OF SECURITY PLAN; THE SECURITY PROGRAM AUDIT, SECURITY ORGANIZATION, PROTECTED AREA PHYSICAL BARRIERS, AND LIGHTING; ACCESS CONTROL (IDENTIFICATION, AUTHORIZATION, AND BADGING); ACCESS CONTROL (SEARCH); ALARM STATIONS; TESTING AND MAINTENANCE; COMPENSATORY MEASURES; RECORDS AND REPORTS; FOLLOW-UP ON PREVIOUS ITEMS OF NONCOMPLIANCE, DEVIATIONS, AND FOLLOW-UP ON IE CIRCULAR 79-16. NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

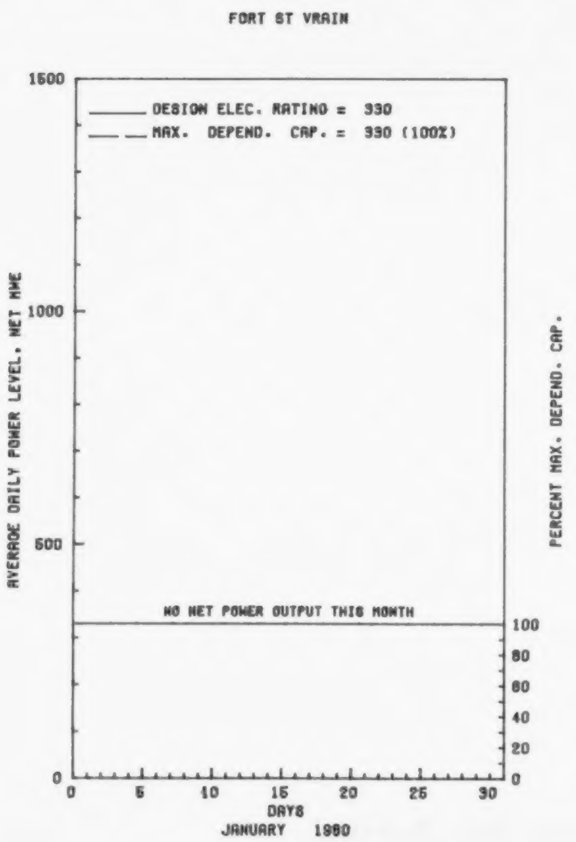
INSPECTION DECEMBER 1-28, 1979 (79-22): ROUTINE, ANNOUNCED INSPECTION OF RESERVE SHUTDOWN SYSTEM; RADIATION PROTECTION; SURVEILLANCE; PART 21 REPORT; MAINTENANCE; PHYSICAL SECURITY; PLANT OPERATIONS; TRAINING; REQUALIFICATION TRAINING; FOLLOWUP ON ITEMS OF NONCOMPLIANCE; AND FOLLOWUP ON INSPECTOR IDENTIFIED AND UNRESOLVED PROBLEMS. WITHIN THE ELEVEN (11) AREAS INSPECTED, TWO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED (INFRACTION -FAILURE TO FOLLOW PROCEDURES (TWO EXAMPLES), AND INFRACTION - FAILURE TO WRITE A PROCEDURE AS REQUIRED BY TECHNICAL SPECIFICATIONS.

REVIEWED BY OMPA LOEB: T. C. Cintula **DATE:** 3-19-80

REVIEWED BY NRR DOR: **DATE:** 3/19/80

FORT ST. VRAIN

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-267
2. Utility Contact: J. W. Gahm (303) 785-6300
3. Licensed Thermal Power (MWt): 842
4. Nameplate Rating (Gross MWe): 403 x 0.85 = 343
5. Design Electrical Rating (Net MWe): 330
6. Maximum Dependable Capacity (Gross MWe): 342
7. Maximum Dependable Capacity (Net MWe): 330
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): 231
10. Reasons for Restrictions, If Any:
NRC RESTRICT (70%) PEND RESOLUTION OF TEMP FLUCTUATIONS

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	744.0	5,161.0
12. Hours Reactor Was Critical	36.0	36.0	2,563.8
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	.0	.0	982.2
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	406	406	478,361
17. Gross Electrical Energy (MWH)	0	0	140,796
18. Net Electrical Energy (MWH)	0	0	123,584
19. Unit Service Factor	.0	.0	19.0
20. Unit Availability Factor	.0	.0	19.0
21. Unit Capacity Factor (MDC Net)	.0	.0	7.3
22. Unit Capacity Factor (DER Net)	.0	.0	7.3
23 Unit Forced Outage Rate	.0	.0	65.1

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
PLANT SHUTDOWN TO REPLACE HELIUM CIRCULATOR.
25. If Shut Down At End of Report Period, Estimated Date of Startup: 04/01/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
79-14	10/26/79	S	744.0	B	4				SCHEDULED SHUTDOWN FOR PLANT MAINTENANCE AND INSTALLATION OF REGION CONSTRAINT DEVICES WAS COMPLETED DECEMBER 24, 1979. ALTHOUGH THE REACTOR WAS CRITICAL AT LESS THAN 2% FOR A BRIEF PERIOD IN JANUARY, DISCOVERY OF A RUPTURED HELIUM CIRCULATOR STATIC SEAL NECESSITATED PLANT SHUTDOWN TO REPLACE THE CIRCULATOR WITH A SPARE. REPORT SUBMITTED ON LER #80-01/01-T-0.

SUMMARY

THE UNIT REMAINED SHUTDOWN THROUGHOUT THE REPORT PERIOD FOR PLANT MAINTENANCE AND INSTALLATION OF REGION CONSTRAINT DEVICES.

LEGEND

[1] Type:
F–Forced
S–Scheduled

[2] Reason:
A–Equipment Failure
B–Maintenance or Test
C–Refueling
D–Regulatory Restriction
E–Operator Training & License Examination
F–Administrative
G–Operational Error
H–Other

[3] Method:
1–Manual
2–Manual Scram
3–Automatic Scram
4–Continuations
5–Load Reduction
9–Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

GINNA

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1. Location (state/county) (nearest population center)	New York/Wayne 15 miles NE of Rochester
2. Electric Reliability Council	NPCC
3. Type of Reactor	PWR
4. Power Level (MWT)	1520
5. Design Electrical Rating (MWE Net)	470
6. Date of Initial Criticality	November 8, 1969
7. Date of Commercial Operation	July 1970
8. Condenser Cooling Method	Once Through
9. Condenser Cooling Water Source	Lake Ontario

Utility & Contractor Information

10. Licensee	Rochester Gas & Electric Corporation
11. Corporate Address	89 East Avenue Rochester, New York
12. Corporate Contact	Leon D. White, Jr., Vice President Electric and Steam Production
13. Architect/Engineer	Gilbert
14. Nuclear Steam Supply System	Westinghouse
15. Constructor	Bechtel

Regulatory Information

16. IE Region responsible	I
17. IE Branch Chief	E. J. Brunner
18. IE Senior Inspector	R. Keimi
19. IE Principal Inspector	R. S. Markowski
20. Licensing Project Manager	J. J. Shea
21. Docket Number	50-244
22. License Number/Date of Issue	DPR-18, 9/19/69
23. Public Document Room	Rochester Public Library Business and Social Science Division 115 South Avenue Rochester, New York 14604

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

- Evaluations of linear indications on pressurizer power operated relief valve yokes are continuing.

FACILITY ITEMS (PLANS AND PROCEDURES):

- Plant shutdown for the annual refueling and maintenance outage has been rescheduled to begin on March 28, 1980.

MANAGERIAL ITEMS:

None.

PLANT STATUS:

- Routine power operation continues.

LAST IE SITE INSPECTION DATE: 1/17/80

- INSPECTION REPORT NO: 50-244/80-01

SUMMARY

INSPECTION SUMMARY

- 50-244/79-17 - Nov 13-16: Routine, unannounced inspection of of plant operations; observation of control room activities; and, onsite Licensee Event Report followup. The inspection involved 25 inspector-hours onsite by one NRC regional based inspector. No items of noncompliance were identified.
- 50-244/79-18 - Dec 3-6: Special unannounced inspection by regional based inspectors of the licensee's response to IE Bulletin 79-19, "Packaging of Low-Level Radioactive Waste for Transport and Burial". Areas inspected included: regulatory requirements, burial site requirements, procedures, training audits, inspection of packages and review of records. In addition, the inspector reviewed actions taken by the licensee in response to previous inspection findings and to the indications of a primary to secondary steam generator leak that occurred on December 1, 1979.
- 50-244/79-19 - Dec 18-20: Routine, unannounced inspection of plant operations; inoffice and onsite review of Licensee Event Reports; review of interim corrective measures associated with a potential failure mechanism associated with the pressurizer power operated relief valve yoke and review of monthly operating reports. The inspection involved 22 inspector-hours onsite by one NRC regional based inspector. No items of noncompliance were identified.

REPORTS RECEIVED FROM LICENSEE

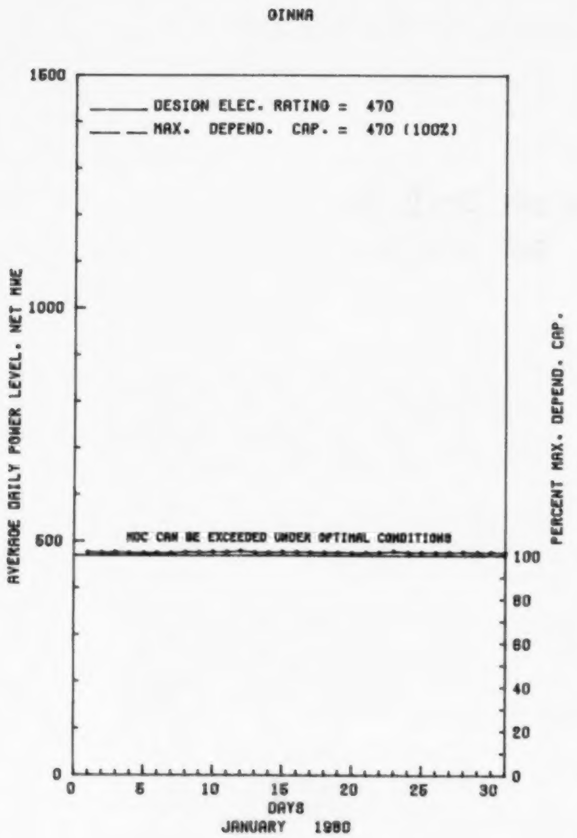
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-022/ 01T	12/01/79	12/14/79	14 Day LER	B Steam Generator tube leak
79-023/ 01T	12/07/79	12/21/79	14 Day LER	Pressurizer PORV relief nozzle indications
79-024/ 03L	12/17/79	12/28/79	30 Day LER	Boric acid storage tanks out of specification for weight percent boric acid
79-025/ 03L	12/23/79	01/22/80	30 Day LER	Surveillance testing of containment air lock door seals not conducted within Technical Specification required interval

REVIEWED BY OMPA LOEB: T. C. Cintula **DATE:** 3-19-80

REVIEWED BY NRR DOR: **DATE:** 3/21/80

GINNA

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

CAPACITY FACTORS
CALCULATED WITH A
WEIGHTED AVERAGE

1. Docket Number: 50-244
2. Utility Contact: A. E. McNamara (716) 546-2700 x291-205
3. Licensed Thermal Power (MWt): 1520
4. Nameplate Rating (Gross MWe): 608 x 0.85 = 517
5. Design Electrical Rating (Net MWe): 470
6. Maximum Dependable Capacity (Gross MWe): 490
7. Maximum Dependable Capacity (Net MWe): 470
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	744.0	83,256.0
12. Hours Reactor Was Critical	744.0	744.0	63,423.8
13. Reactor Reserve Shutdown Hours	.0	.0	1,621.8
14. Hours Generator On-Line	744.0	744.0	66,700.8
15. Unit Reserve Shutdown Hours	.0	.0	8.5
16. Gross Thermal Energy (MWH)	1,127,568	1,127,568	89,463,370
17. Gross Electrical Energy (MWH)	372,061	372,061	29,057,852
18. Net Electrical Energy (MWH)	354,198	354,198	27,515,746
19. Unit Service Factor	100.0	100.0	74.7
20. Unit Availability Factor	100.0	100.0	74.7
*21. Unit Capacity Factor (MDC Net)	101.3	101.3	68.0
22. Unit Capacity Factor (DER Net)	101.3	101.3	68.0
23. Unit Forced Outage Rate	.0	.0	9.8

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
*REFUELING & MAINTENANCE OVERHAUL TENTATIVELY MARCH 28, 1980.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
NONE									

SUMMARY

OPERATION WAS AT FULL POWER THROUGHOUT THE REPORT PERIOD, WITH NO OUTAGES OR REDUCTIONS REPORTED.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

HADDAM NECK

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Connecticut/Middlesex 13 miles E of Meriden
2.	Electric Reliability Council	NPCC
3.	Type of Reactor	PWR
4.	Power Level (MWT)	1825
5.	Design Electrical Rating (MWE Net)	575
6.	Date of Initial Criticality	July 24, 1967
7.	Date of Commercial Operation	January 1, 1968
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Connecticut River

Utility & Contractor Information

10.	Licensee	Connecticut Yankee Atomic Power Co.
11.	Corporate Address	P. O. Box 270 Hartford, Connecticut
12.	Corporate Contact	D. Switzer, President
13.	Architect/Engineer	Stone & Webster
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Senior Inspector	R. Keimig
19.	IE Principal Inspector	W. J. Lazarus
20.	Licensing Project Manager	R. D. Silver
21.	Docket Number	50-213
22.	License Number/Date of Issue	DPR-61, 12/27/74
23.	Public Document Room	Russell Library 119 Broad Street Middletown, Connecticut 06457

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO 10 CFR 20.101 ON FEBRUARY 15, 1979, THREE INDIVIDUALS WERE PERMITTED TO RECEIVE EXPOSURE TO THE WHOLE BODY IN EXCESS OF 1 1/4 REMS DURING THE FIRST QUARTER OF 1979 WITHOUT HAVING THEIR ACCUMULATED OCCUPATIONAL DOSE TO THE WHOLE BODY DETERMINED ON FORM NRC-4, OR ON A CLEAR AND LEGIBLE RECORD CONTAINING ALL THE INFORMATION REQUIRED IN THAT FORM OR OTHERWISE COMPLIED WITH THE REQUIREMENTS OF 10 CFR 20.102, "DETERMINATION OF ACCUMULATED DOSE."
(7906 INFRACTION)

CONTRARY TO 10 CFR 20.201(B) SURVEYS ADEQUATE TO ASSURE COMPLIANCE WITH 10 CFR 20.101 WERE NOT MADE: ON FEBRUARY 3 ON ENTRY TO THE STEAM GENERATOR TO DETERMINE BETA DOSE RATE AND EXPOSURE TO THE LENS OF THE EYE; ON FEBRUARY 16 WHEN PERSONNEL WORKED IN CLOSE PROXIMITY TO A FUEL HANDLING TOOL W/DOSE RATES UP TO 600 MRAD/HR AT CONTACT; AND ON MARCH 7, 1979, WHEN IT WAS FOUND THAT THE SURVEY INSTRUMENT USED TO MONITOR RWP 792395 (CLEAN OUT FLUX THIMBLES) PRODUCED INACCURATE AND UNRELIABLE INFORMATION.
(7906 INFRACTION)

10 CFR 20.103, "EXPOSURE OF INDIVIDUALS TO CONCENTRATIONS OF RADIOACTIVE MATERIALS IN AIR IN RESTRICTED AREAS," STATES IN PART, "(B)(2)... OTHER PRECAUTIONARY PROCEDURES, SUCH AS INCREASED SURVEILLANCE, LIMITATION OF WORKING TIMES, OR PROVISION OF RESPIRATORY PROTECTIVE EQUIPMENT, SHALL BE USED TO MAINTAIN INTAKE OF RADIOACTIVE MATERIAL BY ANY INDIVIDUAL WITHIN ANY PERIOD OF SEVEN CONSECUTIVE DAYS AS FAR BELOW THAT INTAKE OF RADIOACTIVE MATERIAL WHICH WOULD RESULT FROM INHALATION OF SUCH MATERIAL FOR 40 HOURS AT THE UNIFORM CONCENTRATIONS SPECIFIED IN APPENDIX B, TABLE I, COLUMN 1 AS IS REASONABLY ACHIEVABLE." CONTRARY TO THIS REQUIREMENT, BETWEEN FEBRUARY 11 AND MARCH 9, 1979 (OTHER) PRECAUTIONARY PROCEDURES WERE NOT USED TO MAINTAIN INTAKE OF RADIOACTIVE MATERIALS PURSUANT TO 10 CFR 20.103(B)(2).
(7906 INFRACTION)

10 CFR 20.103, "EXPOSURE OF INDIVIDUALS TO CONCENTRATIONS OF RADIOACTIVE MATERIALS IN AIR IN RESTRICTED AREAS," STATES IN PART, "(A)(1) NO LICENSEE SHALL POSSESS, USE, OR TRANSFER LICENSED MATERIAL IN SUCH A MANNER AS TO PERMIT ANY INDIVIDUAL IN A RESTRICTED AREA TO INHALE A QUANTITY OF RADIOACTIVE MATERIAL IN ANY PERIOD OF ONE CALENDAR QUARTER GREATER THAN THE QUANTITY WHICH WOULD RESULT FROM INHALATION FOR 40 HOURS PER WEEK FOR 13 WEEKS AT UNIFORM CONCENTRATIONS OF RADIOACTIVE MATERIAL IN AIR SPECIFIED IN APPENDIX B, TABLE I, COLUMN 1..." IN THIS REGARD SECTION (A)(3) STIPULATES, "FOR PURPOSES OF DETERMINING COMPLIANCE WITH THE REQUIREMENTS OF THIS SECTION THE LICENSEE SHALL USE SUITABLE MEASUREMENTS OF CONCENTRATIONS OF RADIOACTIVE MATERIALS IN AIR FOR DETECTING AND EVALUATING AIRBORNE RADIOACTIVITY IN RESTRICTED AREAS..." CONTRARY TO THIS REQUIREMENT, BETWEEN JANUARY 29 AND MARCH 1, 1979 SUITABLE MEASUREMENTS OF CONCENTRATIONS OF RADIOACTIVE MATERIALS IN AIR FOR DETECTING AND EVALUATING AIRBORNE RADIOACTIVITY WERE NOT MADE IN THE REACTOR CONTAINMENT IN THAT NONE OF THE 910 AIR SAMPLES COLLECTED DURING THIS PERIOD WERE REPRESENTATIVE OF THE CONCENTRATIONS OF AIRBORNE RADIOACTIVITY IN THE BREATHING ZONE OF WORKERS, BUT RATHER, THE AIR SAMPLES WERE COLLECTED IN THE GENERAL AREA OF THE REACTOR CONTAINMENT AT VARIOUS LOCATIONS THAT WERE NOT DIRECTLY REPRESENTATIVE OF THE BREATHING ZONE OF WORKERS.
(7906 INFRACTION)

CONTRARY TO 10 CFR 20.203(E) ON FEBRUARY 15, 1979, THE SPENT FUEL BUILDING AND AREAS OR ROOMS WITHIN THAT BUILDING, IN WHICH LICENSED MATERIAL WAS USED OR STORED CONTAINING RADIOACTIVE MATERIAL IN EXCESS OF 10 TIMES THE QUANTITY OF SUCH MATERIAL SPECIFIED IN APPENDIX C OF THIS PART WAS NOT CONSPICUOUSLY POSTED WITH A SIGN OR SIGNS BEARING THE RADIATION CAUTION SYMBOL AND THE WORDS: CAUTION - RADIOACTIVE MATERIAL. RADIOACTIVE MATERIAL STORED IN THAT LOCATION CONSISTED OF 1) THREE PRESSURIZER CODE SAFETY VALVES HAVING RADIATION LEVELS AS HIGH AS 400 MREM PER HOUR; AND 2) EXPENDED FUEL ASSEMBLIES CONTAINING SEVERAL THOUSAND CURIES OF RADIOACTIVE MATERIAL.
(7906 DEFICIENCY)

CONTRARY TO 10 CFR 20.203(F) ON FEBRUARY 15, 1979, THREE OPEN 55-GALLON DRUMS CONTAINING PRESSURIZER CODE SAFETY VALVES FROM WHICH RADIATION WAS MEASURED TO BE AS HIGH AS 400 MREM PER HOUR AT CONTACT, AND HAVING LOOSE SURFACE CONTAMINATION AS HIGH AS 300,000 DPM/100CM2 DID NOT BEAR ANY LABEL OR OTHER INFORMATION SUFFICIENT TO PERMIT INDIVIDUALS HANDLING OR USING THE CONTAINERS OR WORKING IN THE VICINITY THEREOF, TO TAKE PRECAUTIONS TO AVOID OR MINIMIZE EXPOSURES.
(7906 INFRACTION)

CONTRARY TO 10 CFR 20.405, AS OF MARCH 8, 1979 THE LICENSEE FAILED TO REPORT TO THE COMMISSION AND TO AT LEAST FIVE INDIVIDUALS, ALL OF THE EXPOSURE INFORMATION THAT HAD BEEN DETERMINED FOR INCLUSION ON THE INDIVIDUALS' FORM NRC-5, IN THAT ONLY WHOLE BODY EXPOSURE WAS REPORTED FOR THESE INDIVIDUALS WHEN IN FACT THE INDIVIDUALS WERE DETERMINED TO HAVE RECEIVED EXPOSURE TO SKIN OF THE WHOLE BODY AND HANDS.
(7906 DEFICIENCY)

CONTRARY TO 10 CFR 20.201(B) SURVEYS ADEQUATE TO ASSURE COMPLIANCE WITH 10 CFR 20.101 WERE NOT MADE: ON FEBRUARY 3 WHEN ENTRY WAS MADE INTO NO. 1 STEAM GENERATOR W/250 MREM/HR. (GAMMA) AND 3,000 MRAD/HR (BETA) CONTAMINATION PRESENT; AND ON FEBRUARY 27, 1979 WHEN WORK WAS CONDUCTED ON A VALVE BONNET GASKET READING AS HIGH AS 200 MREM/HR AT CONTACT.
(7906 INFRACTION)

TECHNICAL SPECIFICATION 6.8, PROCEDURES, STATES IN PART, "WRITTEN PROCEDURES AND ADMINISTRATIVE POLICIES SHALL BE ESTABLISHED, IMPLEMENTED AND MAINTAINED THAT MEET OR EXCEED THE REQUIREMENTS AND RECOMMENDATIONS OF SECTIONS 5.1 AND 5.3 OF ANSI N18.7-1972 AND APPENDIX "A" OF USAEC REGULATORY GUIDE 1.33..." USAEC REGULATORY GUIDE 1.33, QUALITY ASSURANCE PROGRAM REQUIREMENTS APPENDIX "A", SECTION G.5.C SPECIFIES PROCEDURES FOR "SURVEYS AND MONITORING". CONTRARY TO THIS REQUIREMENT, AS OF FEBRUARY 14, 1979 THERE WERE NO WRITTEN PROCEDURES ESTABLISHED, IMPLEMENTED OR MAINTAINED THAT ADDRESSED DETECTION, MONITORING, SURVEYS AND EVALUATION OF ALPHA AND BETA RADIOACTIVITY IN REGARDS TO INTERNAL AND EXTERNAL PERSONNEL EXPOSURE, EVEN THOUGH INDIVIDUALS WERE SUBJECT TO EXPOSURE FROM SOURCES OF THESE TYPES OF RADIATION.
(7906 INFRACTION)

CONTRARY TO TECHNICAL SPECIFICATION 6.11, THE FOLLOWING FAILURES TO FOLLOW PROCEDURES WERE IDENTIFIED: 1. ADMINISTRATIVE PROCEDURE NO. 1.1-40, REV. 2 WAS NOT FOLLOWED IN THAT A WORKER WAS PERMITTED TO EXCEED THE ADMINISTRATIVE RADIATION EXPOSURE LIMIT OF 2,750 MREM IN THE FIRST QUARTER, 1979, (RECEIVING AN EXPOSURE OF 2,930 MREM). 2. ADMINISTRATIVE PROCEDURE NO. ACP 1.1-37, SECTION 5.6, WAS NOT FOLLOWED WHEN INDIVIDUALS WORKING UNDER RWP'S FOR EDDY CURRENT TESTING, SLUDGE LANCING, FUEL SHUFFLING, AND REACTOR COOLANT PUMP OIL SWITCH WORK FAILED TO ACKNOWLEDGE THE RESPECTIVE RWP REQUIREMENTS BY INITIALING.
(7906 INFRACTION)

CONTRARY TO TECHNICAL SPECIFICATION 6.13, PARAGRAPH 2, ON FEBRUARY 14 AND 15, 1979, THE FOLLOWING AREAS OF THE PRIMARY AUXILIARY BUILDING WERE NOT LOCKED AND HAD DOSE RATES GREATER THAN 1,000 MREM/HR: RESIDUAL HEAT REMOVAL ACCESS, PIPE TRENCH ACCESS IN BLOWDOWN ROOM, PIPE TRENCH ACCESS IN VALVE OPERATING ROOM AND THE PIPE TRENCH ACCESS IN FAB PASSAGEWAY. ON FEBRUARY 15, 1979, THE FOLLOWING AREAS IN THE REACTOR CONTAINMENT ALSO HAD DOSE RATES GREATER THAN 1,000 MREM/HR AND WERE NOT LOCKED; THE INNER ANNULUS ACCESS, THE LOWER LEVEL OF CONTAINMENT AND THE INNER ANNULUS ACCESS FROM THE CHARGING FLOOR OF CONTAINMENT. - SEVERAL OF THESE AREAS WERE ALSO NOT POSTED AS HIGH RADIATION AREAS.
(7906 INFRACTION)

CONTRARY TO TECHNICAL SPECIFICATION 6.13, PARAGRAPH 1, ON FEBRUARY 15, 1979, TWO INDIVIDUALS PERFORMING HYDRO-LANCING WORK ON NO. 2 STEAM GENERATOR IN ACCORDANCE WITH RADIATION WORK PERMIT (RWP) NO. 791307 WERE WORKING IN A HIGH RADIATION AREA HAVING DOSE RATES BETWEEN 100 AND 3,000 MREM/HR, BUT WERE NOT PROVIDED WITH ANY OF THE CONTROLS SPECIFIED IN TS 6.13.1. RWP NO. 791307 REQUIRED THE USE OF A CONTINUOUS DOSE RATE INDICATING INSTRUMENT.
(7906 INFRACTION)

CONTRARY TO 10 CFR 19.12, INDIVIDUALS WORKING IN THE RESTRICTED AREA BETWEEN FEBRUARY 5 AND MARCH 1, 1979 (PARTICULARLY THOSE INDIVIDUALS WORKING IN THE REACTOR CONTAINMENT TO SUPPORT STEAM GENERATOR EDDY CURRENT TESTING, REACTOR CAVITY AND INNER ANNULUS DECONTAMINATION, AND STEAM GENERATOR HYDROLASING) WERE NOT ADEQUATELY INSTRUCTED IN THE HEALTH PROTECTION PROBLEMS ASSOCIATED WITH EXPOSURE TO RADIOACTIVE MATERIALS OR RADIATION AND IN PRECAUTIONS OR PROCEDURES TO MINIMIZE EXPOSURE IN THAT THE EXTENT OF THESE INSTRUCTIONS WERE NOT COMMENSURATE WITH THE RADIOLOGICAL HEALTH PROTECTION PROBLEMS THAT WERE ENCOUNTERED BY THESE INDIVIDUALS IN THE COURSE OF THEIR WORK.
(7906 INFRACTION)

10 CFR 71.3, "REQUIREMENT FOR LICENSE" STATES, "NO LICENSEE SUBJECT TO THE REGULATIONS IN THIS PART SHALL (A) DELIVER ANY LICENSED MATERIALS TO A CARRIER FOR TRANSPORT OR (B) TRANSPORT LICENSED MATERIAL EXCEPT AS AUTHORIZED IN A GENERAL LICENSE OR SPECIFIC LICENSE ISSUED BY THE COMMISSION, OR AS EXEMPTED IN THIS PART." CONTRARY TO THE ABOVE, ON SEPTEMBER 19, 1979, THE LICENSEE DELIVERED APPROXIMATELY 9.0 CURIES OF RADIOACTIVE WASTE MATERIAL TO A CARRIER FOR DISPOSAL WITHOUT AUTHORIZATION OF A GENERAL OR SPECIFIC LICENSE AND NO EXEMPTION TO 10 CFR 71 WAS APPLICABLE. SPECIFICALLY THE LICENSEE DID NOT HAVE COPIES OF ALL DOCUMENTS REFERENCED BY THE CERTIFICATE OF COMPLIANCE AS REQUIRED BY 10 CFR 71.12(B) AND ALSO THE RADIOACTIVE WASTE HAD NOT BEEN ANALYZED TO DETERMINE THE MOST RESTRICTIVE TRANSPORT GROUP AS REQUIRED BY 10 CFR 71(P)(III).
(7919 INFRACTION)

SEE CONTINUATION PAGE H-3A

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

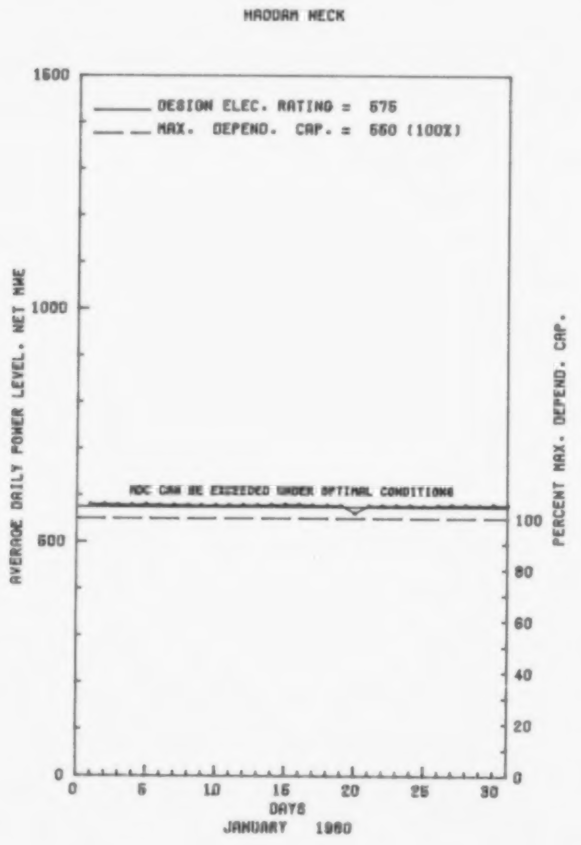
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-006E	12/16/79	12/26/79	Other	An unplanned radioactive gas release occurred that was in excess of Technical Specification limits
80-001/01T	01/03/80	01/15/80	24 Hours	During a routine operability test the sequence timers for the diesel generators were found outside the acceptance criteria
80-002/03L	12/24/79	01/18/80	30 Day LER	Diesel generator 2A out of service when neutral leads were inadvertently grounded during maintenance

REVIEWED BY OMPA LOEB: T. C. Cintula **DATE:** 3-18-80

REVIEWED BY NRR DOR: RJ Silver **DATE:** 3/21/80

HADDAM NECK

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

CAPACITY FACTORS
CALCULATED WITH A
WEIGHTED AVERAGE

1. Docket Number: 50-213
2. Utility Contact: C. Kokoszka (203) 267-2556
3. Licensed Thermal Power (MWt): 1825
4. Nameplate Rating (Gross MWe): 667 x 0.9 = 600
5. Design Electrical Rating (Net MWe): 575
6. Maximum Dependable Capacity (Gross MWe): 577
7. Maximum Dependable Capacity (Net MWe): 550
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	744.0	105,936.0
12. Hours Reactor Was Critical	744.0	744.0	91,960.7
13. Reactor Reserve Shutdown Hours	.0	.0	1,152.6
14. Hours Generator On-Line	744.0	744.0	87,684.2
15. Unit Reserve Shutdown Hours	.0	.0	369.9
16. Gross Thermal Energy (MWH)	1,356,098	1,356,098	151,497,247
17. Gross Electrical Energy (MWH)	450,221	450,221	49,894,687
18. Net Electrical Energy (MWH)	429,984	429,984	47,304,747
19. Unit Service Factor	100.0	100.0	82.8
20. Unit Availability Factor	100.0	100.0	83.1
21. Unit Capacity Factor (MDC Net)	105.1	105.1	82.8
22. Unit Capacity Factor (DER Net)	100.5	100.5	77.0
23. Unit Forced Outage Rate	.0	.0	7.2

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
REFUELING, MAY 1980, 8 TO 12 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	NONE								

SUMMARY

OPERATION WAS AT FULL POWER DURING THE REPORT PERIOD, WITH NO SIGNIFICANT OUTAGES OR REDUCTIONS.

LEGEND

[1] Type:
F–Forced
S–Scheduled

[2] Reason:
A–Equipment Failure
B–Maintenance or Test
C–Refueling
D–Regulatory Restriction
E–Operator Training & License Examination
F–Administrative
G–Operational Error
H–Other

[3] Method:
1–Manual
2–Manual Scram
3–Automatic Scram
4–Continuations
5–Load Reduction
9–Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

None

FACILITY ITEMS (PLANS AND PROCEDURES):

None

MANAGERIAL ITEMS:

PLANT STATUS:

Nothing significant to report.

* LAST IE SITE INSPECTION DATE: 1/21-25/80

* INSPECTION REPORT NO: 50-213/80-02

SUMMARY

INSPECTION SUMMARY

* 50-213/79-06 - Feb 14-16, 26-28 and Mar 5-9: Routine, unannounced inspection by three region-based inspectors (112 hours) of the Radiation Protection Program during the 1979 refueling outage, including Radiation Protection Procedures, Advance Planning and Preparation, Training, Exposure Control, Posting and Control, Radioactive and Contaminated Material Control, and Surveys. During the course of this inspection, allegations concerning the adequacy of the Radiation Protection program were made. These were reviewed in this inspection and in the ensuing investigation (79-07). Nine items of noncompliance were identified. (Failure to control high radiation areas between 100 and 1000 mrem/hr; Failure to control high radiation areas greater than 1000 mrem/hr; Failure to perform surveys; Failure to use precautionary procedures to limit exposure to airborne radioactivity; Failure to instruct workers; Failure to limit personnel exposure; Failure to adhere to procedures; Failure to label containers of radioactive material; and Failure to adequately report workers exposure).

* 50-213/79-07 - Mar 5-9, 16 and 20-23: Investigation by one investigator (81 hours) covering several allegations relating to the Radiation Protection Program during the 1979 refueling outage. The allegations were made in writing by a member of the contractor health physics group and by several members of the licensee and contractor health physics staff during the course of the investigation. Items of noncompliance identified are in inspection Report 79-06.

* 50-213/79-17 - Oct 15-19 and 24-26: Routine, unannounced inspection by two region-based inspectors (84 hours) of: the Security Plan; Security Program Audit; Security Organization; Physical Barriers; Lighting; Access Control; Alarm Stations; Detection Aids; Assessment Aids; Communications; Testing and Maintenance; Compensatory Measures; Power Supply; Response; Locks, Keys and Combinations; Records and Reports and a followup of previous items of noncompliance; followup on I&E Bulletin 79-16, and Independent Actions. No items of noncompliance were identified.

NRC POWER RESTRICTIONS: NONE

HATCH 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Georgia/Appling 11 miles N of Baxley
2.	Electric Reliability Council	Member of SE Electric Reliability Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2436
5.	Design Electrical Rating (MWE Net)	777
6.	Date of Initial Criticality	September 12, 1974
7.	Date of Commercial Operation	December 31, 1975
8.	Condenser Cooling Method	Once Through With Forced Draft Cooling Towers
9.	Condenser Cooling Water Source	Altamaha River

Utility & Contractor Information

10.	Licensee	Georgia Power Company
11.	Corporate Address	270 Peachtree Street, NE Atlanta, Georgia 30303
12.	Corporate Contact	J. H. Miller Executive Vice President
13.	Architect/Engineer	Southern Services, Inc.
14.	Nuclear Steam Supply System	GE
15.	Constructor	Georgia Power Company

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. Lewis
18.	IE Senior Inspector	H. C. Dance
19.	IE Resident Inspector	R. F. Rogers
20.	Licensing Project Manager	J. N. Hannon
21.	Docket Number	50-321
22.	License Number/Date of Issue	DPR-57, 8/6/74
23.	Public Document Room	Appling County Public Library Parker Street Baxley, Georgia 31513

INSPECTION STATUS

ENFORCEMENT STATUS

AS REQUIRED BY CRITERION V OF APPENDIX B TO 10 CFR 50 AND IMPLEMENTED BY HATCH QA MANUAL SECTION 5. "ACTIVITIES AFFECTING QUALITY SHALL BE PRESCRIBED BY DOCUMENTED INSTRUCTIONS, PROCEDURES...AND SHALL BE ACCOMPLISHED IN ACCORDANCE WITH THOSE INSTRUCTIONS, PROCEDURES...". PARAGRAPH E OF HATCH PROCEDURE HNP-1-10124 REQUIRES PIPING SURVEILLANCE TEAMSTO DOCUMENT DIMENSIONAL DISCREPANCIES BETWEEN AS-BUILT CONDITIONS AND DRAWINGS BY MARKING UP THE DRAWINGS. CONTRARY TO THE ABOVE, THE FOLLOWING DIMENSIONAL DISCREPANCIES WERE EITHER NOT DOCUMENTED OR IMPROPERLY DOCUMENTED ON THE MARKED UP DRAWINGS: (1) AN 13-FEET 11-INCH DIMENSION BETWEEN DATA POINT 105 AND THE TEE NEXT TO VALVE F049 ON ISOMETRIC DRAWING E41-102 IS ACTUALLY APPROXIMATELY 15-FEET 11-INCHES AS-BUILT; (2) HANGER H9 SHOWN ON ISOMETRIC DRAWING E41-102 AS 5-FEET 8-INCHES FROM DATA POINT 70 AND ON THE FAR SIDE OF AN EXPANSION JOINT IS ACTUALLY APPROXIMATELY 3-FEET FROM DATA POINT 70 AND ON THE NEAR SIDE OF THE EXPANSION JOINT; (3) SNUBBER M23 SHOWN ON ISOMETRIC DRAWING E51-103 ATTACHED TO AN 8-INCH TEE IS ACTUALLY ATTACHED TO A SPOOLPIECE APPROXIMATELY ONE (1) FOOT FROM THE TEE; (4) AN EXISTING SMALL DIAMETER BRANCH LINE WAS NOT SHOWN ON ISOMETRIC DRAWING E51-103 BETWEEN DATA POINTS 20 AND 30. THE LOCATION MARKED UP ON THE DRAWING BY THE SURVEILLANCE TEAM IS APPROXIMATELY 4-FEET FROM THE ACTUAL LOCATION.
(7931 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

PLANT OPERATING AT FULL POWER.

* LAST IE SITE INSPECTION DATE: FEBRUARY 25-28, 1980

* INSPECTION REPORT NO: 50-321/80-07

SUMMARY

INSPECTION SUMMARY

* INSPECTION NOVEMBER 10 - DECEMBER 7 (79-37): THIS INSPECTION INVOLVED 40 INSPECTOR-HOURS ON SITE OF TECHNICAL SPECIFICATION COMPLIANCE, REPORTABLE OCCURRENCES, HOUSEKEEPING, OPERATOR PERFORMANCE, OVERALL PLANT OPERATIONS, QUALITY ASSURANCE PRACTICES, STATION AND CORPORATE MANAGEMENT PRACTICES, CORRECTIVE AND PREVENTATIVE MAINTENANCE ACTIVITIES, SITE SECURITY PROCEDURES, RADIATION CONTROL ACTIVITIES, SURVEILLANCE ACTIVITIES, AND FOLLOWUP OF PREVIOUS INSPECTION FINDINGS. OF THE TWELVE AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

REPORTS RECEIVED FPOM LICENSEE

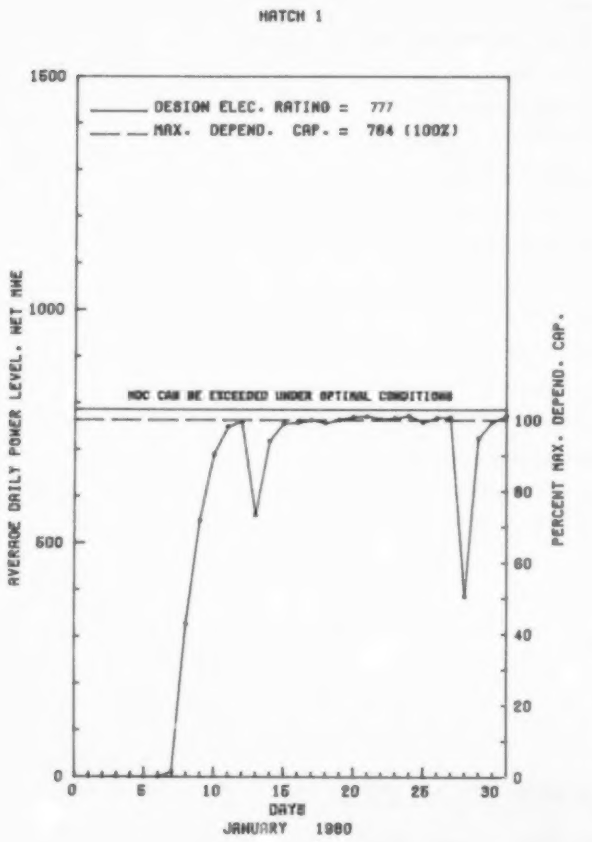
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-91/ 03L-0	11/06/79	11/27/79	30 DAY LER	PENETRATION FIRE BARRIER NOT INSTALLED.
79-092/ 03L-0	11/07/79	12/05/79	30 DAY LER	LATE REPORT OF LICENSEE EVENT.
79-094/ 03L-0	11/21/79	12/07/79	30 DAY LER	PLANT SERVICE WATER PUMP FAILURE.
79-097/ 03L-0	11/28/79	12/11/79	30 DAY LER	DRYWELL ATMOSPHERE INSTRUMENT ERROR.
79-098/ 01T-0	12/10/79	12/17/79	14 DAY LER	REACTOR CORE ISOLATION COOLING SYSTEM INOPERABILITY NOT RECOGNIZED.
79-099/ 03L-0	12-06/79	12/27/79	30 DAY LER	STACK INLET VALVE INOPERABLE.
79-100/ 03L-0	12/11/79	12/20/79	30 DAY LER	LOW PRESSURE COOLANT INJECTION ISOLATION VALVE LEAKING.
79-101/ 03L-0	12/12/79	01/03/80	30 DAY LER	DIESEL GENERATOR FAILED TO START.
79-104/ 03L-0	12/16/79	01/02/80	30 DAY LER	STANDBY SERVICE WATER PUMP FAILURE.
79-105/ 01T-0	12/28/79	01/04/80	14 DAY LER	TORUS VENT LINE DOWNCOMER BRACE MISSING.
79-106/ 03L-0	12/16/79	01/08/80	30 DAY LER	HIGH PRESSURE COOLANT INJECTION INOPERABLE.
80-001/ 01T-0	01/03/80	01/14/80	14 DAY LER	PIPE RESTRAINT MISSING PIN.

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 3-19-80

REVIEWED BY NRR DOR: John N Hannon **DATE:** 3-21-80

HATCH 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-321
2. Utility Contact: J. Roden (912) 367-7781
3. Licensed Thermal Power (MWt): 2436
4. Nameplate Rating (Gross MWe): 1000 x 0.85 = 850
5. Design Electrical Rating (Net MWe): 777
6. Maximum Dependable Capacity (Gross MWe): 797
7. Maximum Dependable Capacity (Net MWe): 764
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: DER=777.3 PER LICENSEE COMMUNICATION OF 2/29/80
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	744.0	35,809.0
12. Hours Reactor Was Critical	630.4	630.4	26,408.9
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	581.1	581.1	24,849.5
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,279,160	1,279,160	51,632,195
17. Gross Electrical Energy (MWH)	424,930	424,930	16,695,870
18. Net Electrical Energy (MWH)	405,788	405,788	15,867,721
19. Unit Service Factor	78.1	78.1	69.4
20. Unit Availability Factor	78.1	78.1	69.4
21. Unit Capacity Factor (MDC Net)	71.4	71.4	58.0
22. Unit Capacity Factor (DER Net)	69.4	69.4	56.4
23. Unit Forced Outage Rate	21.9	21.9	16.4

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-01	12/13/79	F	162.9	A	4		HA	GENERA	SHUTDOWN DUE TO GROUND FAULT IN THE MAIN GENERATOR ROTOR.
80-02	01/08/80	F	.0	A	5		HA	INSTRU	REDUCTION DUE TO FAULTY GENERATOR CORE MONITOR AND STARTUP TESTING.
80-03	01/12/80	S	.0	B	5		ZZ	CONROD	REDUCTION TO PERFORM WEEKLY TURBINE TEST AND DO ROD PATTERN ADJUSTMENT.
80-04	01/27/80	F	.0	H	5		XX	PUMPXX	REDUCTION DUE TO LOSS OF CORE FLOW WHEN "A" RECIRC PUMP TRIPPED.

SUMMARY

THE UNIT REMAINED SHUTDOWN FOR REPAIRS TO THE GENERATOR MOTOR UNTIL 01/07/80. OPERATION CONTINUED AT NEAR FULL POWER THE REMAINDER OF THE PERIOD, WITH VERY BRIEF REDUCTIONS TO REPAIR A GENERATOR CORE MONITOR, FOR TURBINE TESTING AND CONTROL ROD ADJUSTMENT, AND FOLLOWING A RECIRCULATION PUMP TRIP.

LEGEND

[1] Type:
F–Forced
S–Scheduled

[2] Reason:
A–Equipment Failure
B–Maintenance or Test
C–Refueling
D–Regulatory Restriction
E–Operator Training & License Examination
F–Administrative
G–Operational Error
H–Other

[3] Method:
1–Manual
2–Manual Scram
3–Automatic Scram
4–Continuations
5–Load Reduction
9–Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

HATCH 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Georgia/Appling 11 miles N of Baxley
2.	Electric Reliability Council	Member of SE Electric Reliability Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2436
5.	Design Electrical Rating (MWE Net)	784
6.	Date of Initial Criticality	July 4, 1978
7.	Date of Commercial Operation	September 5, 1979
8.	Condenser Cooling Method	Cooling Towers
9.	Condenser Cooling Water Source	Altamaha River

Utility & Contractor Information

10.	Licensee	Georgia Power Company
11.	Corporate Address	270 Peach Street, NE Atlanta, Georgia 30303
12.	Corporate Contact	J. H. Miller Executive Vice President
13.	Architect/Engineer	Southern Services, Inc./Bechtel
14.	Nuclear Steam Supply System	GE
15.	Constructor	Georgia Power Company

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. Lewis
18.	IE Senior Inspector	H. C. Dance
19.	IE Resident Inspector	R. F. Rogers
20.	Licensing Project Manager	J. N. Hannon
21.	Docket Number	50-366
22.	License Number/Date of Issue	NPF-5/June 13, 1978
23.	Public Document Room	Appling County Public Library Parker Street Baxley, Georgia 31513

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

PLANT OPERATING AT FULL POWER.

+ LAST IE SITE INSPECTION DATE: FEBRUARY 25-28, 1980

+ INSPECTION REPORT NO: 50-366/80-07

SUMMARY

INSPECTION SUMMARY

+ INSPECTION NOVEMBER 10 - DECEMBER 7 (79-41): THIS INSPECTION INVOLVED 40 INSPECTOR-HOURS ON SITE OF TECHNICAL SPECIFICATION COMPLIANCE, REPORTABLE OCCURRENCES, HOUSEKEEPING, OPERATOR PERFORMANCE, OVERALL PLANT OPERATIONS, QUALITY ASSURANCE PRACTICES, STATION AND CORPORATE MANAGEMENT PRACTICES, CORRECTIVE AND PREVENTATIVE MAINTENANCE ACTIVITIES, SITE SECURITY PROCEDURES, RADIATION CONTROL ACTIVITIES, SURVEILLANCE ACTIVITIES, AND FOLLOWUP OF PREVIOUS INSPECTION FINDINGS. OF THE TWELVE AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN ELEVEN AREAS; ONE APPARENT ITEM OF NONCOMPLIANCE WAS FOUND IN ONE AREA (INFRACTION - COMMENCING WORK PRIOR TO APPROVAL OF A RADIATION WORK PERMIT).

REPORTS RECEIVED FROM LICENSEE

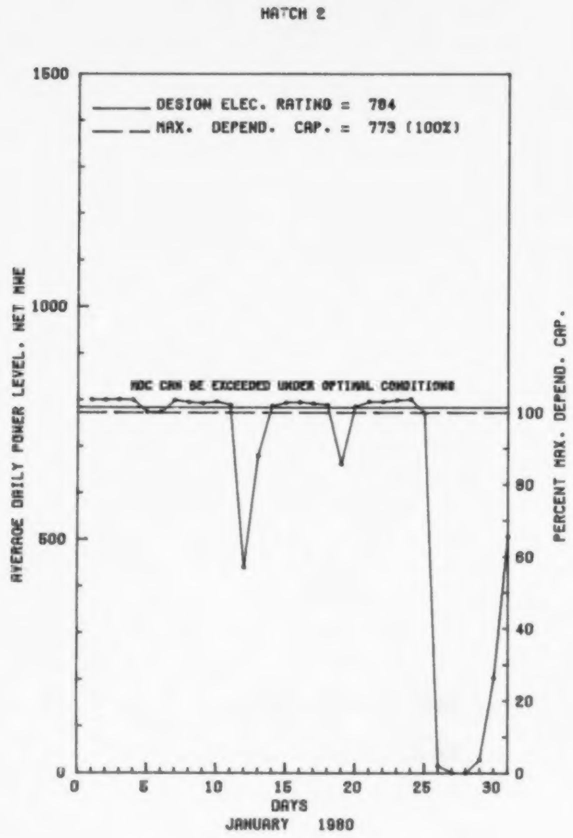
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-121/ 03L-0	11/17/79	11/30/79	30 DAY LER	MABLHR LIMIT EXCEEDED.
79-122/ 03L-0	11/17/79	12/07/79	30 DAY LER	LOW PRESSURE COOLANT INJECTION VALVE POWER SUPPLY DEGRADED.
79-123/ 03L-0	11/17/79	12/06/79	30 DAY LER	PLANT SERVICE WATER PUMP TEST FAILURE.
79-124/ 03L-0	11/20/79	12/06/79	30 DAY LER	TORUS/DRYWELL VACUUM BREAKER INOPERABLE.
79-127/ 03L-0	11/27/79	12/06/79	30 DAY LER	HIGH PRESSURE COOLANT INJECTION INSTRUMENT SET-POINT DRIFT.
79-128/ 03L-0	11/28/79	12/06/79	30 DAY LER	DRYWELL ATMOSPHERE RECORDER INOPERABLE.
79-130/ 03L-0	12/05/79	12/26/79	30 DAY LER	REACTOR WATER LEVEL INSTRUMENT DRIFT.
79-131/ 03L-0	12/06/79	12/28/79	30 DAY LER	TORUS HIGH WATER INSTRUMENT FAILURE.
79-133/ 03L-0	12/11/79	12/27/79	30 DAY LER	HIGH PRESSURE COOLANT INJECTION ISOLATED.
79-134/ 03L-0	12/14/79	12/28/79	30 DAY LER	FISSION PRODUCT MONITOR FAILURE.
79-135/ 03L-0	12/19/79	01/08/80	30 DAY LER	RCIC LEAR DETECTION FAILURE.
79-136/ 03L-0	12/16/79	01/08/80	30 DAY LER	TORUS VACUUM BREAKER INOPERABLE.
79-137/ 03L-0	12/24/79	01/08/80	30 DAY LER	DRYWELL TEMPERATURE RECORDER INOPERABLE.
79-138/ 01T-0	12/28/79	12/31/79	14 DAY LER	HIGH PRESSURE COOLANT INJECTION STEAM LINE INSTRUMENT DRIFT.
80-001/ 03L-0	01/02/80	01/15/80	30 DAY LER	TORUS WATER LEVEL TRANSMITTER DRIFT.

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 3-20-80

REVIEWED BY NRR DOR: John N. Hannon **DATE:** 3-21-80

HATCH 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-366
2. Utility Contact: J. Roden (912) 367-7781
3. Licensed Thermal Power (MWt): 2436
4. Nameplate Rating (Gross MWe): 1000 x .85 = 850
5. Design Electrical Rating (Net MWe): 784
6. Maximum Dependable Capacity (Gross MWe): 806
7. Maximum Dependable Capacity (Net MWe): 773
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: CHANGED NET MDC TO REFLECT NEW UTILITY RATINGS
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	744.0	3,577.0
12. Hours Reactor Was Critical	684.1	684.1	3,347.6
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	661.2	661.2	3,075.9
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,471,890	1,471,890	6,965,952
17. Gross Electrical Energy (MWH)	499,790	499,790	2,335,750
18. Net Electrical Energy (MWH)	478,796	478,796	2,235,933
19. Unit Service Factor	88.9	88.9	86.0
20. Unit Availability Factor	88.9	88.9	86.0
21. Unit Capacity Factor (MDC Net)	83.3	83.3	80.9
22. Unit Capacity Factor (DER Net)	82.8	82.8	79.7
23. Unit Forced Outage Rate	.0	.0	8.8

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): VENT HEADER DEFLECTOR AND SURVEIL - 03/01 - 04/13/80.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-01	01/11/80	S	.0	H	5		ZZ	CONROD	REDUCTION IN POWER TO PERFORM TURBINE TEST AND ROD SEQUENCE EXCHANGE.
80-02	01/26/80	S	81.4	D	1		SA	ZZZZZZ	SHUTDOWN TO PERFORM PRIMARY CONTAINMENT ISOLATION MODIFICATIONS.
80-03	01/29/80	S	1.4	B	9		ZZ	TURBIN	UNIT WAS BROUGHT OFF LINE TO PERFORM TURBINE OVERSPEED TESTING.

SUMMARY

OPERATION WAS ROUTINE DURING THE REPORT PERIOD, WITH A LOAD REDUCTION ON 01/11/80 FOR TURBINE TESTING AND CONTROL ROD ADJUSTMENT. OUTAGES OCCURRED ON 01/26/80 FOR PRIMARY CONTAINMENT ISOLATION MODIFICATIONS, AND ON 01/29/80 FOR TURBINE OVERSPEED TESTING.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

HUMBOLDT BAY

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	California/Humboldt 4 miles S of Eureka
2.	Electric Reliability Council	WSCC
3.	Type of Reactor	BWR
4.	Power Level (MWT)	220
5.	Design Electrical Rating (MWE Net)	65
6.	Date of Initial Criticality	February 16, 1963
7.	Date of Commercial Operation	August 1963
8.	Condenser Cooling Method	Circulating sea water
9.	Condenser Cooling Water Source	Pacific Ocean (Humboldt Bay)

Utility & Contractor Information

10.	Licensee	Pacific Gas & Electric Co.
11.	Corporate Address	77 Beale Street San Francisco, California 94106
12.	Corporate Contact	P. A. Crane, Jr. Asst. General Counsel
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	GE
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	V
17.	IE Branch Chief	J. L. Crews
18.	IE Senior Inspector	D. M. Sternberg
19.	IE Principal Inspector	J. D. Carlson
20.	Licensing Project Manager	V. L. Rooney
21.	Docket Number	50-133
22.	License Number/Date of Issue	DPR-7, 8/28/62
23.	Public Document Room	Humboldt County Library 636 F Street Eureka, California 95501

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST (CITE RO REPORT NUMBER, IF APP.)

SYSTEMS AND COMPONENT PROBLEMS:

None.

FACILITY ITEMS (PLANS AND PROCEDURES):

The plant has been shut down since July 1976.

Several items remaining to be completed during the outage have been deferred pending resolution of the seismic issues at the plant site. A date for return to operations has not been established.

MANAGERIAL ITEMS:

None

PLANT STATUS:

Plant is shutdown.

LAST IE SITE INSPECTION DATE: 12/18-20/79

INSPECTION REPORT NO: 50-133/79-04

SUMMARY

INSPECTION SUMMARY

+ No inspections conducted in January 1980.

LICENSE STATUS

Order issued 5/21/76 requiring Utility to prove seismic qualification before resuming operation of unit after outage of 7/2/76.

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
NONE				

REVIEWED BY OMPA LOEB: ______ DATE: 3·18·80

REVIEWED BY NRR DOR: ______ DATE: 3/20/80

HUMBOLDT BAY

AVERAGE DAILY POWER LEVEL (MWe)

OPERATING STATUS

1. Docket Number: 50-133
2. Utility Data Prepared By: B. C. Getty (707) 443-2787
3. Licensed Thermal Power (MWt): 220
4. Nameplate Rating (Gross MWe): 77 X 0.85 = 65
5. Design Electrical Rating (Net MWe): 65
6. Maximum Dependable Capacity (Gross MWe): 0
7. Maximum Dependable Capacity (Net MWe): 0
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:

None

9. Power Level To Which Restricted, If Any (Net MWe): None
10. Reasons For Restrictions, If Any

NO DECISION ON FUTURE OPERATION
SHUTDOWN INDEFINITELY

This

11. Hours In Reporting Period
12. Number Of Hours Reactor Was Cri
13. Reactor Reserve Shutdow
14. Hours Generator
15. Unit R
1 rd)
erated (MWH)
Net gy Generated (MWH)
19. Unit S ce Factor
20. Unit Availability Factor
21. Unit Capacity Factor (Using MDC Net) 1/
22. Unit Capacity Factor (Using DER Net) 1/
23. Unit Forced Outage Rate
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

None

25. If Shut Down At End Of Report Period, Estimated Date of Startup: UNKNOWN

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
SEE SUMMARY BELOW									

SUMMARY

THE UNIT WAS SHUTDOWN ON 7/2/76 FOR SEISMIC MODIFICATIONS. THE UNIT REMAINS IN THE COLD SHUTDOWN CONDITION.

LEGEND

[1] Type:
F-Forced
S-Scheduled

[2] Reason:
A-Equipment Failur
B-Maintenance or Te
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error
H-Other

[3] Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Continuations
5-Load Reduction
9-Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

INDIAN POINT 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	New York/Westchester 25 miles N of New York City
2.	Electric Reliability Council	NPCC
3.	Type of Reactor	PWR/fossil superheat
4.	Power Level (MWT)	615
5.	Design Electrical Rating (MWE Net)	265
6.	Date of Initial Criticality	August 2, 1962
7.	Date of Commercial Operation	October 1962
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Hudson River

Utility & Contractor Information

10.	Licensee	Consolidated Edison Company of New York, Inc.
11.	Corporate Address	4 Irving Place New York, New York
12.	Corporate Contact	W. Cahill, Vice President
13.	Architect/Engineer	Utility
14.	Nuclear Steam Supply System	B&W
15.	Constructor	Utility

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Senior Inspector	R. Keimig
19.	IE Resident Inspector	T. Rebelowski (914-739-9360) FTS 656-9220
20.	Licensing Project Manager	L. Olshan x27144
21.	Docket Number	50-03
22.	License Number/Date of Issue	DPR-5, 3/26/62
23.	Public Document Room	White Plains Public Library 100 Martine Avenue White Plains, New York 10601

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

```
    I N S P E C T I O N   S T A T U S

    OTHER ITEMS OF INTEREST

    SYSTEMS AND COMPONENT PROBLEMS:

    None

    FACILITY ITEMS (PLANS AND PROCEDURES):

*   Order to Show Cause why the operating provision of
    License No. DPR-5 should not be revoked and why
    the licensee should not submit a plan to decommission
    the facility was issued on February 11, 1980.

    MANAGERIAL ITEMS:

    None

    PLANT STATUS.

    Defueled.

  LAST IE SITE INSPECTION DATE:  7/24-27/79

*         INSPECTION REPORT NO:  50-03/79-02

3.  SUMMARY

    INSPECTION SUMMARY

    None.
```

LICENSE STATUS

The unit produced electricity to the expiration date (October 31, 1974) of delay granted by the AEC for installation of an ECCS system. The ECCS system was not subsequently installed and the unit was defueled on January 30, 1976.

On February 11, 1980, the NRC requested the utility to show cause as to why the operating authority of the license should not be revoked and why the licensee should not submit a plan within 120 days of this order to decommission the unit.

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
NONE				

REVIEWED BY OMPA LOEB: T. C. Cwtula DATE: 3·19·80

REVIEWED BY NRR DOR: ____________ DATE: ____________

INDIAN POINT 1

AVERAGE DAILY POWER LEVEL (MWe)

OPERATING STATUS

1. Docket Number: 50-003
2. Utility Data Prepared By: L. J. Kewula (914)-694-6000 x209
3. Licensed Thermal Power (MWt): 615
4. Nameplate Rating (Gross MWe): 275
5. Design Electrical Rating (Net MWe): 265
6. Maximum Dependable Capacity (Gross MWe): 0
7. Maximum Dependable Capacity (Net MWe): 0
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons: None
9. Power Level To Which Restricted, If Any (Net MWe): None
10. Reasons For Restrictions, If Any:

	This Month	Yr. to Date
11. Hours In Reporting Period		
12. Number Of Hours Reactor Was Critical		
13. Reactor Reserve Shutdown Hours		
14. Hours Generator On Line		
15. Unit Reserve Shutdown Ho		
16. Gross Thermal E		
17. Gross		
20. Unit A		
21. Unit Capacity Factor (Net) 1/		
22. Unit Capacity Factor (Using DER Net) 1/		
23. Unit Forced Outage Rate		

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): None

25. If Shut Down At End Of Report Period, Estimated Date of Startup: N/A

NO DECISION ON FUTURE OPERATION
SHUTDOWN INDEFINITELY
ALL FUEL UNLOADED.

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
SEE SUMMARY BELOW									

SUMMARY

UNIT SHUTDOWN ON OCTOBER 31, 1974, ALL FUEL HAS BEEN UNLOADED; NO DECISION ON FUTURE OPERATION HAS BEEN MADE.

LEGEND

[1] Type:
- F-Forced
- S-Scheduled

[2] Reason:
- A-Equipment Failure
- B-Maintenance or Test
- C-Refueling
- D-Regulatory Restriction
- E-Operator Training & License Examination
- F-Administrative
- G-Operational Error
- H-Other

[3] Method:
- 1-Manual
- 2-Manual Scram
- 3-Automatic Scram
- 4-Continuations
- 5-Load Reduction
- 9-Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source

INDIAN POINT 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1. Location (state/county) (nearest population center)	New York/Westchester 25 miles N of New York City
2. Electric Reliability Council	NPCC
3. Type of Reactor	PWR
4. Power Level (MWT)	2758
5. Design Electrical Rating (MWE Net)	873
6. Date of Initial Criticality	May 22, 1973
7. Date of Commercial Operation	August 1973
8. Condenser Cooling Method	Once Through
9. Condenser Cooling Water Source	Hudson River

Utility & Contractor Information

10. Licensee	Consolidated Edison Company of New York, Inc.
11. Corporate Address	4 Irving Place New York, New York
12. Corporate Contact	W. J. Cahill, Jr., Vice President
13. Architect/Engineer	UE&C
14. Nuclear Steam Supply System	Westinghouse
15. Constructor	J. A. Jones/UE&C

Regulatory Information

16. IE Region responsible	I
17. IE Branch Chief	E. J. Brunner
18. IE Senior Inspector	R. Keimig
19. IE Resident Inspector	T. Rebelowski (914-739-9380) FTS 656-9220
20. Licensing Project Manager	L. Olshan (X27144)
21. Docket Number	50-247
22. License Number/Date of Issue	DPR-26, 10/19/71
23. Public Document Room	White Plains Public Library 100 Martine Avenue White Plains, New York 10601

INSPECTION STATUS

ENFORCEMENT STATUS

FAILURE TO RECOGNIZE THAT QUADRANT POWER TILT CALCULATION RESULT WAS IN EXCESS OF TECH. SPEC. LIMIT.
(7923 DEFICIENCY)

10 CFR 71.3, "REQUIREMENT FOR LICENSE" STATES, "NO LICENSEE SUBJECT TO THE REGULATIONS IN THIS PART SHALL (A) DELIVER ANY LICENSED MATERIALS TO A CARRIER FOR TRANSPORT OR (B) TRANSPORT LICENSED MATERIAL EXCEPT AS AUTHORIZED IN A GENERAL LICENSE OR SPECIFIC LICENSE ISSUED BY THE COMMISSION, OR AS EXEMPTED IN THIS PART." CONTRARY TO THE ABOVE, ON 9/5/79, THE LICENSEE DELIVERED APPROXIMATELY 110 CURIES OF RADIOACTIVE WASTE MATERIAL TO A CARRIER FOR DISPOSAL WITHOUT AUTHORIZATION OF A GENERAL OR SPECIFIC LICENSE, & NO EXEMPTION TO 10 CFR 71 WAS APPLICABLE. SPECIFICALLY, THE LICENSEE DID NOT HAVE COPIES OF ALL DOCUMENTS REFERENCED BY THE CERTIFICATE OF COMPLIANCE AS REQUIRED BY 10 CFR 71.12(B).
(7924 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

- None

FACILITY ITEMS (PLANS AND PROCEDURES):

- Next scheduled refueling outage 1/1/81.

MANAGERIAL ITEMS:

- (1) Confirmatory order issued February 11, 1980 concerning implementation of TMI Lessons Learned and Emergency Preparedness.

  (2) The licensee has appointed a new Instrumentation and Control Supervisor and a new Security Supervisor.

PLANT STATUS:

- Unit 2 returned to service on 2/10 after completion of turbine inspection and repairs.

- LAST IE SITE INSPECTION DATE: 2/2/80

- INSPECTION REPORT NO: 50-247/80-01

SUMMARY

INSPECTION SUMMARY

- 50-247/79-21 - Oct 14-Nov 10: Routine inspection (41 hours by 2 resident inspectors) of plant operations including: shift logs and records; tours of the facility; observations of physical security; Emergency Planning operations; surveillance testing; review of licensee events; and, follow-up on previous inspection items. No items of noncompliance were identified.

- 50-247/79-23 - Nov 11-Dec 8: Routine inspection (25 hours by 2 resident inspectors) of plant operations including: shift logs and records; tours of the facility; observations of physical security; Emergency Planning Drill; Reactor Coolant System Leakage and Pressurizer Safety Valve testing; periodic reports; and changes in station management staff. One item of noncompliance was identified (Failure to recognize a Quadrant Power Tilt calculation result in excess of Technical Specification limits).

- 50-247/79-24 - Nov 15-16: Routine, unannounced inspection of the licensee's response to IE Bulletin 79-19 including maintenance of regulatory requirements, training, audits and procedures. The inspection involved 28 hours onsite by two NRC region-based inspectors. Of the five areas inspected, one item of noncompliance was found (Failure to have copies of all documents referenced by the Certificate of Compliance).

REPORTS RECEIVED FROM LICENSEE

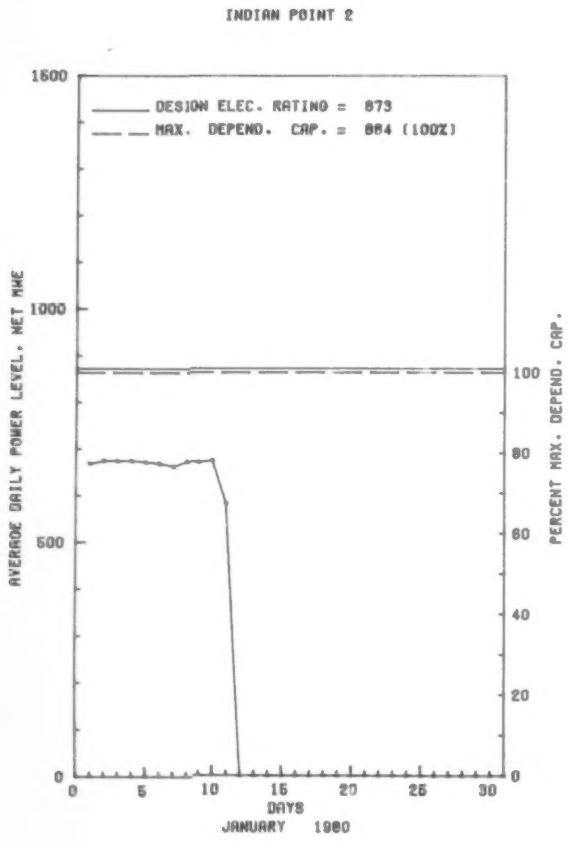
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-023/03L	11/09/79	12/07/79	30 Day LER	Containment isolation valve PCV-1217 would not shut
79-024/03L	12/02/79	12/28/79	30 Day LER	Letdown relief valve no. 203 found leaking

REVIEWED BY OMPA LOEB: T. C. Cintula **DATE:** 3•19•80

REVIEWED BY NRR DOR: __________ **DATE:** __________

INDIAN POINT 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

ITEM 21 CALCULATED BY USING A WEIGHTED AVERAGE

1. Docket Number: 50-247
2. Utility Contact: L. Kawula (914) 694-6000 @ I.P.
3. Licensed Thermal Power (MWt): 2758
4. Nameplate Rating (Gross MWe): 1126 x 0.9 = 1013
5. Design Electrical Rating (Net MWe): 873
6. Maximum Dependable Capacity (Gross MWe): 900
7. Maximum Dependable Capacity (Net MWe): 864
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	744.0	48,995.0
12. Hours Reactor Was Critical	262.2	262.1	32,556.6
13. Reactor Reserve Shutdown Hours	481.8	481.8	723.2
14. Hours Generator On-Line	262.2	262.2	31,651.4
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	660,999	660,999	81,493,525
17. Gross Electrical Energy (MWH)	183,820	183,820	25,316,906
18. Net Electrical Energy (MWH)	171,515	171,515	24,136,615
19. Unit Service Factor	35.2	35.2	64.6
20. Unit Availability Factor	35.2	35.2	64.6
21. Unit Capacity Factor (MDC Net)	26.7	26.7	57.2
22. Unit Capacity Factor (DER Net)	26.4	26.4	56.4
23. Unit Forced Outage Rate	.0	.0	7.7

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 02/11/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
31	01/11/80	S	481.8	D	3				TURBIN UNIT SHUTDOWN FOR L.P. TURBINE INSPECTION.

SUMMARY

OPERATION WAS ROUTINE UNTIL 01/11/80, WHEN THE UNIT WAS SHUTDOWN FOR LOW PRESSURE TURBINE INSPECTION.

LEGEND

[1] Type:
F-Forced
S-Scheduled

[2] Reason:
A-Equipment Failure
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error
H-Other

[3] Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Continuations
5-Load Reduction
9-Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

INDIAN POINT 3

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	New York/Westchester 25 miles N of New York City
2.	Electric Reliability Council	NPCC
3.	Type of Reactor	PWR
4.	Power Level (MWT)	3025
5.	Design Electrical Rating (MWE Net)	965
6.	Date of Initial Criticality	April 6, 1976
7.	Date of Commercial Operation	August 30, 1976
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Hudson River

Utility & Contractor Information

10.	Licensee	Power Authority of the State of New York, Inc.
11.	Corporate Address	10 Columbus Circle New York, New York
12.	Corporate Contact	G.T. Barry General Manager
13.	Architect/Engineer	UE&C
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	WEDCO

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Senior Inspector	R. Keimig
19.	IE Resident Inspector	T. Rebelowski (914-739-9360) FTS 656-9220
20.	Licensing Project Manager	L. Olshan x27144
21.	Docket Number	50-286
22.	License Number/Date of Issue	DPR-64, 12/12/75
23.	Public Document Room	White Plains Public Library 100 Martine Avenue White Plains, New York 10601

INSPECTION STATUS

ENFORCEMENT STATUS

10 CFR 50 APPENDIX B, CRITERION XVI, STATES THAT MEASURES SHALL BE ESTABLISHED TO ASSURE THAT CONDITIONS ADVERSE TO QUALITY ARE PROMPTLY CORRECTED. THE FSAR, APPENDIX D, SECTION D.2.2.8 STATES THAT THE QA PROGRAM COMPLIES WITH WASH 1283, DATED MAY 24, 1974. WASH 1283 ENDORSES ANSI N45.2.12-1974. ANSI N45.2.12-1974 STATES THAT THE AUDITED ORGANIZATION SHALL RESPOND TO AN AUDIT REPORT WITHIN 30 DAYS OF THE RECEIPT OF THE RESULTS OF THE AUDIT. THE RESPONSE SHALL CLEARLY STATE THE CORRECTIVE ACTION TAKEN OR A SCHEDULED DATE BY WHICH CORRECTIVE ACTION WILL BE COMPLETED. CONTRARY TO THE ABOVE, THE ENGINEERING DEPARTMENT HAD NOT RESPONDED TO AUDIT REPORT NO. 1 OF THE IMPLEMENTATION OF ENGINEERING DEPARTMENT PROCEDURES AS OF SEPTEMBER 19, 1979. THE RESULTS OF AUDIT REPORT NO. 1 WERE FORWARDED TO THE ENGINEERING DEPARTMENT ON OCTOBER 5, 1978.
(7920 INFRACTION)

10 CFR 50, APPENDIX B, CRITERION XVII, STATES, IN PART, "...CONSISTENT WITH APPLICABLE REGULATORY REQUIREMENT, THE APPLICANT SHALL ESTABLISH REQUIREMENTS CONCERNING RECORD RETENTION, SUCH AS DURATION, LOCATION, AND ASSIGNED RESPONSIBILITY." AMENDMENT 12 TO THE FACILITY OPERATING LICENSE, WHICH INCORPORATES THE ORIGINAL SUBMITTAL OF CHAPTER 17.2 (MARCH 1, 1977) TO THE FSAR, STATES, IN PART (17.2.2.1), "...THE PROGRAM COMPLIES WITH THE GUIDANCE SET FORTH IN...WASH-1309...DATED MAY 10, 1974..." ANSI N45.2.9-1974 (15 DRAFT, REVISION 0), REQUIREMENTS FOR COLLECTION, STORAGE, AND MAINTENANCE OF QUALITY ASSURANCE RECORDS FOR NUCLEAR POWER PLANTS, IS CONTAINED WITHIN WASH-1309. PARAGRAPH 5.6, FACILITY, DELINEATES THE PHYSICAL STORAGE REQUIREMENTS FOR SINGLE COPY RECORD STORAGE. CONTRARY TO THE ABOVE, THE STORAGE FACILITIES FOR SINGLE COPY RECORDS DOES NOT COMPLY WITH THE CONSTRUCTION REQUIREMENTS OF ANSI N45.2.9, PARAGRAPH 5.6 (E.G., THE TEMPORARY "VAULT," 1 HOUR FIRE RATED CABINETS IN THE SITE WAREHOUSE ADMINISTRATIVE AREA AND CORPORATE RECORD STORAGE FACILITIES).
(7920 INFRACTION)

THE J. A. FITZPATRICK TECHNICAL SPECIFICATIONS, SECTION 6.1 STATES THAT THE RESIDENT MANAGER REPORTS TO THE GENERAL MANAGER AND CHIEF ENGINEER (A SINGLE POSITION). THE INDIAN POINT NO. 3 TECHNICAL SPECIFICATIONS, SECTION 6.2.1 STATES THAT THE ORGANIZATION SHALL BE AS SHOWN ON FIGURE 6.2.1 WHICH SHOWS THE RESIDENT MANAGER REPORTING TO THE GENERAL MANAGER AND CHIEF ENGINEER. CONTRARY TO THE ABOVE, THE RESIDENT MANAGERS AT J. A. FITZPATRICK AND INDIAN POINT NO. 3 WERE REPORTING TO THE DIRECTOR OF POWER OPERATIONS, WHO IS ADMINISTRATIVELY, AT A LOWER LEVEL THAN THE GENERAL MANAGER AND CHIEF ENGINEER. A FORMAL REQUEST FOR A CHANGE IN TECHNICAL SPECIFICATIONS WAS NOT SUBMITTED FOR FITZPATRICK AND NRC APPROVAL WAS NOT GRANTED FOR EITHER FACILITY.
(7920 INFRACTION)

10 CFR 20.401, "RECORDS OF SURVEYS, RADIATION MONITORING, AND DISPOSAL," REQUIRES IN SECTION (B) THAT EACH LICENSEE MAINTAIN RECORDS SHOWING THE RESULTS OF SURVEYS REQUIRED BY 10 CFR 20.201(B). 1. CONTRARY TO THE ABOVE, AS OF OCTOBER 31, 1979, A RECORD OF THE SURVEY MADE BY THE LICENSEE, DURING OCTOBER 1979, TO DETERMINE EXTREMITY PERSONNEL MONITORING REQUIREMENTS FOR INDIVIDUALS ENTERING THE STEAM GENERATORS HAD NOT BEEN MAINTAINED. 2. CONTRARY TO THE ABOVE, AS OF OCTOBER 31, 1979, A RECORD OF THE SURVEY MADE BY THE LICENSEE, DURING OCTOBER 1979, TO DETERMINE COMPLIANCE WITH THE WHOLE BODY DOSE LIMITS OF INDIVIDUALS ENTERING THE STEAM GENERATORS HAD NOT BEEN MAINTAINED.
(7924 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

None

FACILITY ITEMS (PLANS AND PROCEDURES):

+ None

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-013/ 01T	11/27/79	11/28/79	14 Day LER	Refueling surveillance test found pressurizer safety valve out of spec
79-014/ 03L	12/02/79	12/28/79	30 Day LER	Hydrogen recombiner surveillance test indicated high dilute temperature (3PT-R12)
79-015/ 01T	11/27/79	11/28/79	14 Day LER	Refueling surveillance test found accumulator level transmitters out of calibration
79-016/ 01T	11/27/79	12/07/79	14 Day LER	Westinghouse notified licensee of possible nonconservative feature in Appendix K LOCA/ECCS analysis related to fuel burst
79-017/ 03L	12/10/79	01/09/80	30 Day LER	Refueling water storage TK calibration test found low level limit alarm out of specification
79-018/ 03L	11/23/79	12/21/79	30 Day LER	Defective snubbers have been identified during surveillance testing (3PT-R38)

MANAGERIAL ITEMS:

+ Confirmatory Order issued February 11, 1980 concerning implementation of TMI Lessons Learned and Emergency Preparedness.

PLANT STATUS:

+ Refueling operations continued.

+ LAST IE SITE INSPECTION DATE: 1/19/80

+ INSPECTION REPORT NO: 50-286/80-01

SUMMARY

INSPECTION SUMMARY

+ Summaries of the following reports are not yet available at this time and will be provided subsequently.

+ September 30-October 27, 1979	-	50-286/79-23
October 23-25, 1979	-	50-286/79-25
November 13, 25, 1979	-	50-286/79-26
October 28-November 25, 1979	-	50-286/79-27
November 27, 1979	-	50-286/79-28
November 25-December 22, 1979	-	50-286/79-29
December 10-14, 1979	-	50-286/79-30
December 23, 1979	-	50-286/79-31

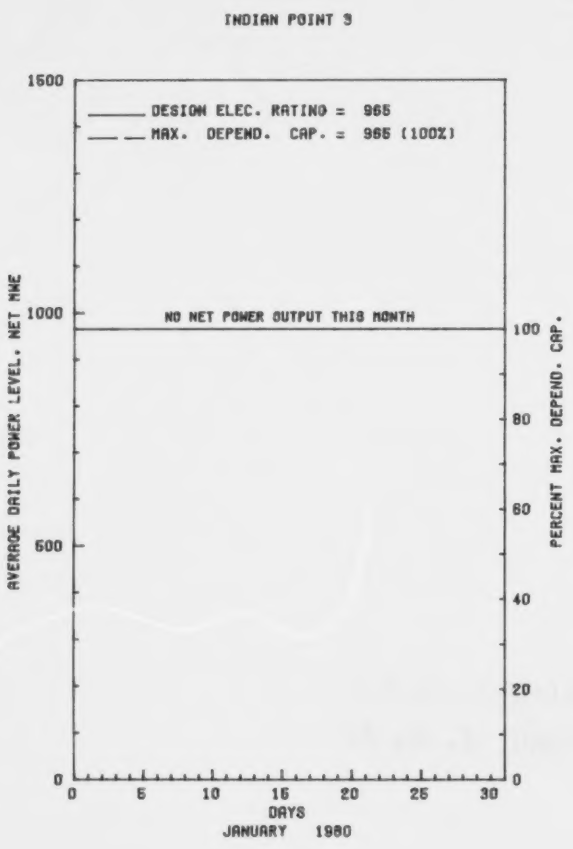
REVIEWED BY OMPA LOEB: T.C. Cintula DATE: 3·19·80

REVIEWED BY NRR DOR: ____________ DATE: ____________

best copy available

INDIAN POINT 3

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-286
2. Utility Contact: C. Connell (914) 739-8200 x242
3. Licensed Thermal Power (MWt): 3025
4. Nameplate Rating (Gross MWe): 1126 x 0.9 = 1013
5. Design Electrical Rating (Net MWe): 965
6. Maximum Dependable Capacity (Gross MWe): 1000
7. Maximum Dependable Capacity (Net MWe): 965
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	744.0	39,001.0
12. Hours Reactor Was Critical	50.7	50.7	21,635.6
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	.0	.0	21,032.8
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	0	56,104,467
17. Gross Electrical Energy (MWH)	0	0	18,365,101
18. Net Electrical Energy (MWH)	0	0	17,643,437
19. Unit Service Factor	.0	.0	70.1
20. Unit Availability Factor	.0	.0	70.1
21. Unit Capacity Factor (MDC Net)	.0	.0	60.9
22. Unit Capacity Factor (DER Net)	.0	.0	60.9
23. Unit Forced Outage Rate	.0	.0	4.1

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 02/05/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
36	09/14/79	S	744.0	C	4				REFUELING OUTAGE CONTINUES.

SUMMARY

THE UNIT REMAINED SHUTDOWN THROUGHOUT THE REPORT PERIOD FOR REFUELING.

LEGEND

[1] Type:
F-Forced
S-Scheduled

[2] Reason:
A-Equipment Failure
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error
H-Other

[3] Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Continuations
5-Load Reduction
9-Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

KEWAUNEE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Wisconsin/Kewaunee 27 miles E of Green Bay
2.	Electric Reliability Council	N/A
3.	Type of Reactor	PWR
4.	Power Level (MWT)	1650
5.	Design Electrical Rating (MWE Net)	535
6.	Date of Initial Criticality	March 7, 1974
7.	Date of Commercial Operation	June 1974
8.	Condenser Cooling Method	Once through condenser and return to Lake Michigan
9.	Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10.	Licensee	Wisconsin Public Service Corp.
11.	Corporate Address	P. O. Box 1200 Green Bay, Wisconsin 54305
12.	Corporate Contact	E. R. Mathews Power Supply & Eng.
13.	Architect/Engineer	Fluro Pioneer, Inc.
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Pioneer Services & Eng. Co.

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Senior Inspector	R. F. Warnick
19.	IE Principal Inspector	N. C. Choules
20.	Licensing Project Manager	R. B. Licciardo X28362
21.	Docket Number	50-305
22.	License Number/Date of Issue	DPR-43 issued 12/21/73
23.	Public Document Room	Kewaunee Public Library 822 Juneau Street Kewaunee, Wisconsin 54216

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENTS

+ As a result of reviews for IE Bulletin 79-14, the licensee determined that three pipe restraints had not been installed on one of two 16-inch service water lines. The restraints were installed within 2 days after they were discovered missing. (79-21)

PLANT STATUS

+ The plant was shutdown from January 17 to 28 following the failure of the reserve auxiliary transformer. The plant was operating at 100% power at the end of the month.

+LAST IE SITE INSPECTION DATE: December 6-7, 1979

+ INSPECTION REPORT NO: 79-22

SUMMARY

INSPECTION SUMMARY

+ Inspection on December 3-7, (79-21): Routine unannounced inspection of design changes, non-routine events, quality assurance program, onsite review committee, IE Bulletins, and independent inspection. The inspection involved 37 inspector hours onsite by one NRC inspector. No items of noncompliance or deviations were identified.

+ Inspection on December 6-7, (79-22): Non-routine, announced inspection of the licensee's response to IE Bulletin No. 79-19. The inspection involved 12 inspector hours onsite by one NRC inspector. No items of noncompliance or deviations were identified.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

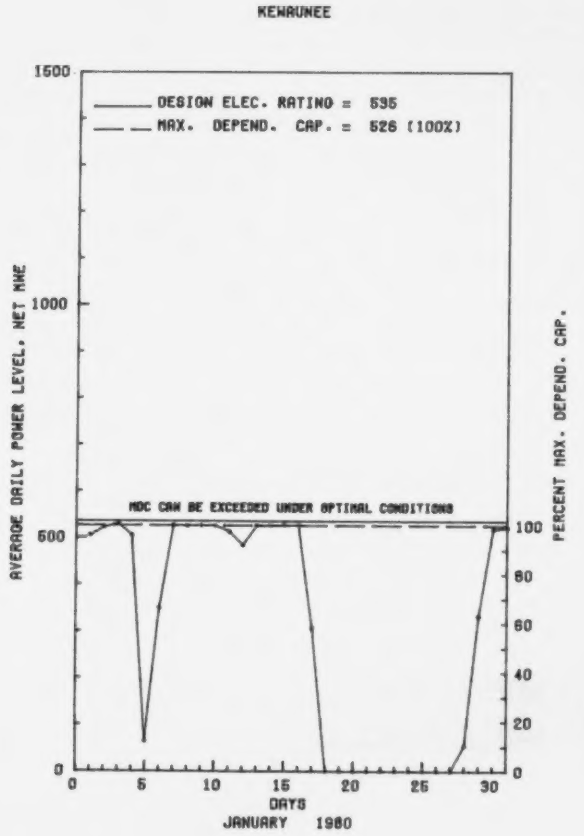
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-28/03L-0	12/07/79	01/07/80	30 Day LER	Three horizontal pipe restraints not installed on a 16 inch service water header.

REVIEWED BY OMPA LOEB: T. C. Cintula DATE: 3.19.80

REVIEWED BY NRR DOR: R. Licciardo DATE: 3.21.80

KEWAUNEE

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

ITEM 21 CALCULATED BY USING A WEIGHTED AVERAGE

1. Docket Number: 50-305
2. Utility Contact: J. Wallace (414) 388-2560 x207
3. Licensed Thermal Power (MWt): 1650
4. Nameplate Rating (Gross MWe): 622 x 0.9 = 560
5. Design Electrical Rating (Net MWe): 535
6. Maximum Dependable Capacity (Gross MWe): 545
7. Maximum Dependable Capacity (Net MWe): 526
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	744.0	49,355.0
12. Hours Reactor Was Critical	479.2	479.2	41,431.7
13. Reactor Reserve Shutdown Hours	152.7	152.7	2,330.5
14. Hours Generator On-Line	463.9	463.9	40,459.9
15. Unit Reserve Shutdown Hours	.0	.0	10.0
16. Gross Thermal Energy (MWH)	713,133	713,133	61,864,256
17. Gross Electrical Energy (MWH)	236,900	236,900	20,420,700
18. Net Electrical Energy (MWH)	225,880	225,880	19,424,958
19. Unit Service Factor	62.4	62.4	82.0
20. Unit Availability Factor	62.4	62.4	82.0
21. Unit Capacity Factor (MDC Net)	57.7	57.7	75.1
22. Unit Capacity Factor (DER Net)	56.7	56.7	73.6
23. Unit Forced Outage Rate	37.6	37.6	5.0

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUELING; MAY 10, 1980; SIX WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
1	01/04/80	F	15.4	A	3		EB	ELECON	1. BUS FAULT FROM MAIN AUXILIARY TRANSFORMER RESULTED IN A TURBINE/REACTOR TRIP.
2A	01/17/80	F	158.7	A	3		EB	TRANSF	2A. BUSHING FAILURE IN RESERVE AUXILIARY TRANSFORMER CAUSED LOSS OF POWER TO ALL BUT SAFEGUARDS BUSSES. UNIT TRIP FOLLOWED IMMEDIATELY.
2B	01/17/80	F	106.0	A	3		CB	MECFUN	2B. APPARENT FAILURE OF NO. 2 SEAL ON 1B RCP EXTENDED THIS OUTAGE.

SUMMARY

THE UNIT ENTERED THE REPORT PERIOD AT REDUCED LOAD FOR TURBINE STOP VALVE TESTS. ON 01/04/80, A BUS FAULT FROM THE MAIN AUXILIARY TRANSFORMER RESULTED IN A TURBINE/REACTOR TRIP. A REACTOR/TURBINE TRIP OCCURRED ON 01/17/80 FROM A BUSHING FAILURE IN THE RESERVE AUXILIARY TRANSFORMER. THE UNIT WAS SHUTDOWN FROM 01/18/80 THROUGH 01/27/80 TO REPAIR THE RESERVE AUXILIARY TRANSFORMER AND A NO. 2 SEAL ON REACTOR COOLANT PUMP B.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram.
3--Automatic Scram.
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

LACROSSE

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Wisconsin/Vernon 19 miles S of LaCrosse
2.	Electric Reliability Council	MARCA
3.	Type of Reactor	BWR
4.	Power Level (MWT)	165
5.	Design Electrical Rating (MWE Net)	50
6.	Date of Initial Criticality	July 11, 1967
7.	Date of Commercial Operation	November 1, 1969
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Mississippi River

Utility & Contractor Information

10.	Licensee	Dairyland Power Cooperative
11.	Corporate Address	2615 East Avenue South La Crosse, Wisconsin
12.	Corporate Contact	F. Linder
13.	Architect/Engineer	Sargent & Lundy
14.	Nuclear Steam Supply System	Allis-Chalmers
15.	Constructor	Maxson Construction

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	D. C. Boyd
18.	IE Senior Inspector	R. F. Warnick
19.	IE Principal Inspector	K. Ridgway
20.	Licensing Project Manager	J. J. Shea
21.	Docket Number	50-409
22.	License Number/Date of Issue	DPR-45 issued 8/28/73
23.	Public Document Room	LaCrosse Public Library 800 Main Street LaCrosse, Wisconsin 54601

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

FACILITY ITEMS(PLANS AND PROCEDURES)

+ Current plans are to shutdown the reactor about February 1, 1980 to shuffle six fuel elements in order to extend cycle 6 lift until May 1, 1980 when refueling will be accomplished. (79-22)

MANAGERIAL ITEMS:

+ The licensee has lost two more licensed operators and is activitely recruiting to fill these vacancies and other engineering positions that are vacant. (79-22)

PLANT STATUS

Plant is operating routinely.

+LAST IE SITE INSPECTION DATE: December 3-6, 1979

+ INSPECTION REPORT NO: 79-22

INSPECTION SUMMARY

+ Inspection on December 3-6, (79-22): Routine unannounced inspection of the licensee's operational activities: and followup actions relative to IE Bulletins, IE Circulars, Licensee Event Reports and special reports. The inspection involved 29 inspector hours onsite by one NRC inspector. No items of noncompliance were identified in the five areas inspected.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

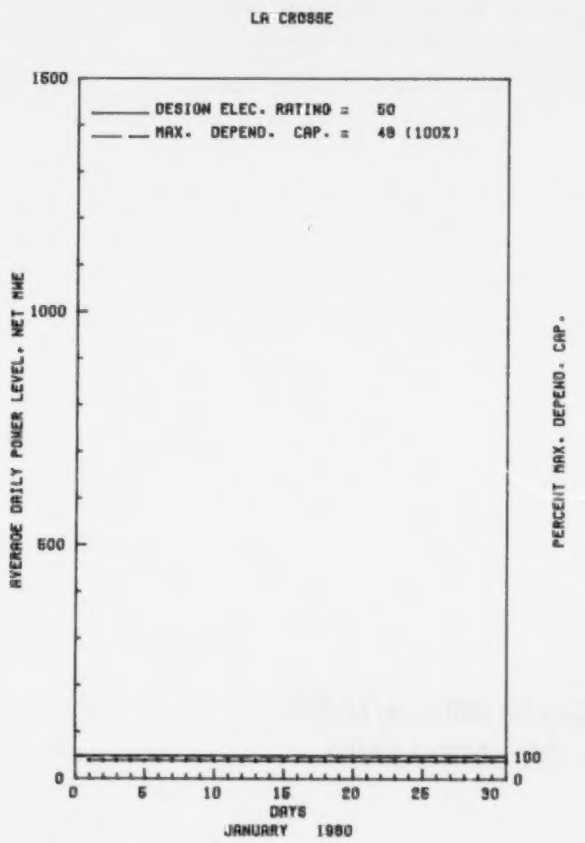
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-18/ 01T-0	12/14/79	12/27/79	14 Day LER	LACBWR received notification from Allis Chalmers Manufacturing Co. that the plant's containment isolation dampers as they are presently installed in the ventilation system require repositioning from full open to an opening of not more than 25 degrees to ensure quick full closure at the maximum containment.
79-19/ 01T-0	12/23/79	12/26/79	14 Day LER	Periodic leakage rate test for the emergency airlock did not meet type B test criteria as specified by TS 5.2.1.2.b.

REVIEWED BY OMPA LOEB: John J. Rizzo DATE: 3-19-80

REVIEWED BY NRR DOR: S. J. Nowicki DATE: 3-21-80

LACROSSE

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-409
2. Utility Contact: L. S. Goodman (608) 689-2331
3. Licensed Thermal Power (MWt): 165
4. Nameplate Rating (Gross MWe): 76.8 x 0.85 =65
5. Design Electrical Rating (Net MWe): 50
6. Maximum Dependable Capacity (Gross MWe): 50
7. Maximum Dependable Capacity (Net MWe): 48
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	744.0	85,859.0
12. Hours Reactor Was Critical	744.0	744.0	58,550.1
13. Reactor Reserve Shutdown Hours	.0	.0	478.0
14. Hours Generator On-Line	744.0	744.0	53,759.0
15. Unit Reserve Shutdown Hours	.0	.0	79.0
16. Gross Thermal Energy (MWH)	104,003	104,003	7,388,782
17. Gross Electrical Energy (MWH)	30,902	30,902	2,230,034
18. Net Electrical Energy (MWH)	29,142	29,142	2,061,877
19. Unit Service Factor	100.0	100.0	59.8
20. Unit Availability Factor	100.0	100.0	59.9
21. Unit Capacity Factor (MDC Net)	81.6	81.6	47.8
22. Unit Capacity Factor (DER Net)	78.3	78.3	45.9
23. Unit Forced Outage Rate	.0	.0	6.3

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
EQUIPMENT INSTALLATION - NUREG-0578. - 03/11/80
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	NONE								

SUMMARY

OPERATION WAS STEADY AT 85% POWER THROUGHOUT THE REPORT PERIOD, IN PREPARATION FOR THE 03-80 OUTAGE TO INSTALL EQUIPMENT PER NUREG-0578.

LEGEND

[1] Type:
F-Forced
S-Scheduled

[2] Reason:
A-Equipment Failure
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error
H-Other

[3] Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Continuations
5-Load Reduction
9-Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

MAINE YANKEE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Maine/Lincoln 10 miles N of Bath (near Wiscasset)
2.	Electric Reliability Council	NPCC
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2630
5.	Design Electrical Rating (MWE Net)	825
6.	Date of Initial Criticality	October 23, 1972
7.	Date of Commercial Operation	December 28, 1972
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Back River (Atlantic)

Utility & Contractor Information

10.	Licensee	Maine Yankee Atomic Power Co.
11.	Corporate Address	20 Turnpike Road Westboro, Massachusetts 01581
12.	Corporate Contact	D. W. Edwards, Assistant to Vice President
13.	Architect/Engineer	Stone & Webster
14.	Nuclear Steam Supply System	Combustion Engineering
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	IE Senior Inspector	R. Keimig
19.	IE Principal Inspector	W. J. Lazarus
20.	Licensing Project Manager	M. B. Fairtile
21.	Docket Number	50-309
22.	License Number/Date of Issue	DPR-36, 9/15/72
23.	Public Document Room	Wiscasset Public Library High Street Wiscasset, Maine 04578

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

- An error identified in the Main Steam Line Break safety analysis will require a modification to feedwater regulating valve controls prior to startup.

FACILITY ITEMS (PLANS AND PROCEDURES):

- None

MANAGERIAL ITEMS:

- Startup following the refueling outage is planned for February 28, 1980.

PLANT STATUS:

- Routine plant operation continued until January 11, when a refueling/maintenance outage commenced.

- LAST IE SITE INSPECTION DATE: 1/25/80

- INSPECTION REPORT NO: 50-309/80-01

SUMMARY

INSPECTION SUMMARY

- 50-309/79-17 - Dec 3-7: Routine unannounced inspection by region-based inspectors of: plant operations including review of shift logs and records, tour of accessible areas, and observation of plant activities; licensee actions concerning selected previous inspection findings and IE Bulletins and Circulars; required licensee periodic reports; and follow-up on selected Licensee Event Reports (LER's). The inspection involved 37 hours onsite by two NRC inspectors. No items of noncompliance were identified.

- 50-309/79-21 - Dec 17-19: Routine, unannounced inspection of the licensee conducted emergency drill; tour of accessible areas; and a followup on selected Radwaste Shipment Discrepancy Reports. The inspection involved 22 hours onsite by two region-based inspectors. No items of noncompliance were identified.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

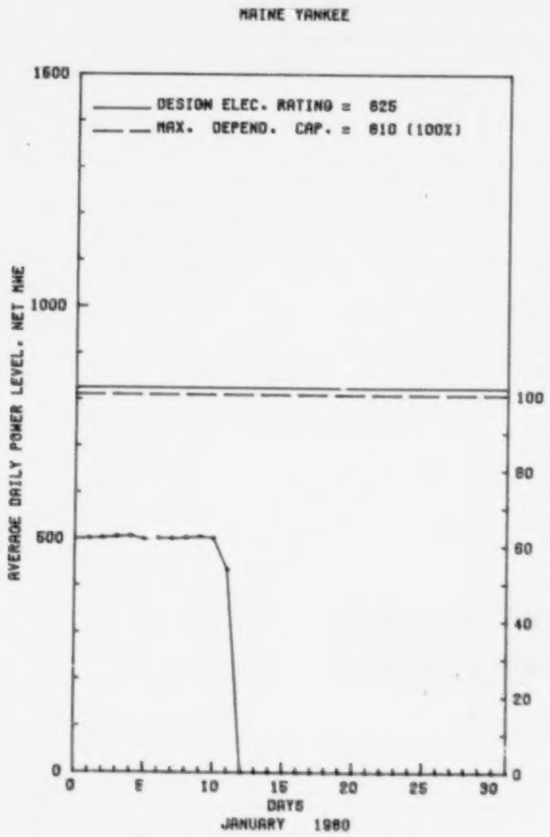
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-029/ 99X	10/25/79	12/07/79	30 Day LER	Reactor Coolant dose equivalent Iodine-131 greater than 1.0 μCi/gm
79-030/ 99X	11/05/79	12/07/79	30 Day LER	Reactor Coolant dose equivalent Iodine-131 greater than 1.0 μCi/gm
79-031/ 03L	12/27/79	01/11/80	30 Day LER	Relay contacts on each of two separate RPS trip matrix relays found not opening reliably during special surveillance testing

REVIEWED BY OMPA LOEB: M R Beebe DATE: 03/19/80

REVIEWED BY NRR DOR: Norton B. Fairtile DATE: 3/20/80

MAINE YANKEE

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

CAPACITY FACTORS CALCULATED WITH A WEIGHTED AVERAGE

1. Docket Number: 50-309
2. Utility Contact: Robert H. Groce (617) 366-9011
3. Licensed Thermal Power (MWt): 2630
4. Nameplate Rating (Gross MWe): 900 x 0.9 = 810
5. Design Electrical Rating (Net MWe): 825
6. Maximum Dependable Capacity (Gross MWe): 850
7. Maximum Dependable Capacity (Net MWe): 810
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): 500
10. Reasons for Restrictions, If Any: L. P. TURBINE STEAM FLOW RESTRICTION.

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	744.0	63,372.6
12. Hours Reactor Was Critical	263.4	263.4	51,472.1
13. Reactor Reserve Shutdown Hours	0	0	0
14. Hours Generator On-Line	263.1	263.1	49,789.8
15. Unit Reserve Shutdown Hours	0	0	0
16. Gross Thermal Energy (MWH)	442,499	442,499	105,674,582
17. Gross Electrical Energy (MWH)	139,660	139,660	34,712,840
18. Net Electrical Energy (MWH)	131,040	131,040	32,958,276
19. Unit Service Factor	35.4	35.4	78.5
20. Unit Availability Factor	35.4	35.4	78.6
21. Unit Capacity Factor (MDC Net)	21.7	21.7	67.6
22. Unit Capacity Factor (DER Net)	21.3	21.3	65.3
23. Unit Forced Outage Rate	0	0	6.9

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 03/05/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
8-79-4	01/11/80	S	481.0	C	1				CYCLE 4/5 REFUELING SHUTDOWN.

SUMMARY

OPERATION WAS ROUTINE IN AN END-OF-CYCLE COASTDOWN UNTIL 01/11/80, WHEN THE REFUELING OUTAGE BEGAN.

LEGEND

[1] Type:
F-Forced
S-Scheduled

[2] Reason:
A-Equipment Failure
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error
H-Other

[3] Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Continuations
5-Load Reduction
9-Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

MILLSTONE 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Connecticut/New London 5 miles SW of New London
2.	Electric Reliability Council	NPCC
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2011
5.	Design Electrical Rating (MWE Net)	660
6.	Date of Initial Criticality	October 26, 1970
7.	Date of Commercial Operation	March 1971
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Long Island Sound

Utility & Contractor Information

10.	Licensee	Northeast Nuclear Energy Company
11.	Corporate Address	P. O. Box 270 Hartford, Connecticut
12.	Corporate Contact	W. G. Counsil, Vice President
13.	Architect/Engineer	Ebasco
14.	Nuclear Steam Supply System	GE
15.	Constructor	Ebasco

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	IE Senior Inspector	E. McCabe (FTS: 244-2000)
19.	IE Resident Inspector	T. Shedlosky (203) 447-3170
20.	Licensing Project Manager	J. J. Shea
21.	Docket Number	50-245
22.	License Number/Date of Issue	DPR-21, 10/7/70
23.	Public Document Room	Waterford Public Library Rope Ferry Road Route 156 Waterford, Connecticut 06385

INSPECTION STATUS

ENFORCEMENT STATUS

ON 11/7/79, AN OPEN CAN OF COMBUSTIBLE DEBRIS WAS OBSERVED IN THE UNIT 1 CONTROL ROOM, & AN OPEN CAN OF COMBUSTIBLE DEBRIS WAS OBSERVED IN THE UNIT 2 D.C SWITCHGEAR ROOM. EVIDENCE OF SMOKING WAS OBSERVED IN THE UNIT 1 CONTROL ROOM MAIN CONTROL BOARD.
(7925 INFRACTION)

CONTRARY TO TS 6.8.1, 6.8.3 & 1.2.A, PROCEDURES WERE NOT ESTABLISHED FOR THREE SURVEILLANCES: TEMPORARY CHANGES WERE MADE TO 2 CALIBRATION PROCEDURES WHICH WERE NOT DOCUMENTED; AND 4 SURVEILLANCE TESTS EXCEEDED THEIR REQUIRED TEST INTERVAL.
(7927 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

- The Isolation Condenser System has been removed from Standby Service (isolated) because of construction deficiencies in the steam line containment penetration pipe restraint. The reactor has been operated at a maximum power level of 40% with the system isolated.

FACILITY ITEMS (PLANS AND PROCEDURES):

None

MANAGERIAL ITEMS:

None

PLANT STATUS:

- The plant has been operating at 40% power with the Isolation Condenser System out of service. Representative offgas and stack release rates are 65,000 microcuries per second and 150 microcuries per second, respectively.

- LAST IE SITE INSPECTION DATE: 1/31/80

- INSPECTION REPORT NO: 50-245/80-01

SUMMARY

INSPECTION SUMMARY

- 50-245/79-25 - Nov 5-9: Routine unannounced inspection (14 hours) of the Fire Protection/Prevention Program including: assignment of functional responsibilities; implementation of administrative controls; fire brigade training; ignition source and combustible materials control; fire protection systems and components; plant modifications; surveillance test records; and critical fire areas. Noncompliance: One (Failure to follow housekeeping procedures).

- 50-245/79-26 - Nov 27-30, Dec 3-7: Routine, unannounced inspection (49 hours) by a region-based inspector and an NRR engineer of the Quality Assurance Program (QAP) implementation in the areas of: QA/QC administration; design changes/modifications; document control; QAP changes; and followup on previously identified items. Noncompliances: None.

- 50-245/79-27 - Nov 27-30: Routine, unannounced inspection by a region-based inspector (30 hours) of: action on previous inspection findings; administrative control of safety related calibrations; calibration of safety-related components and equipment required by Technical Specifications; calibrations required by Technical Specifications of components and equipment associated with safety-related systems and/or functions; calibration and control of test equipment; witnessing of calibrations; technician qualifications; and, control room tour. Noncompliances: One (Failure to establish surveillance procedures, document temporary changes, and adhere to surveillance frequencies).

- 50-245/79-30 - Dec 19-20: Special, unannounced inspection by a region-based inspector (7 hours) of the licensee's response to Bulletin 79-19 including: regulatory requirements, procedures; training; audits; records of shipments; and onsite observations. Noncompliances: None.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

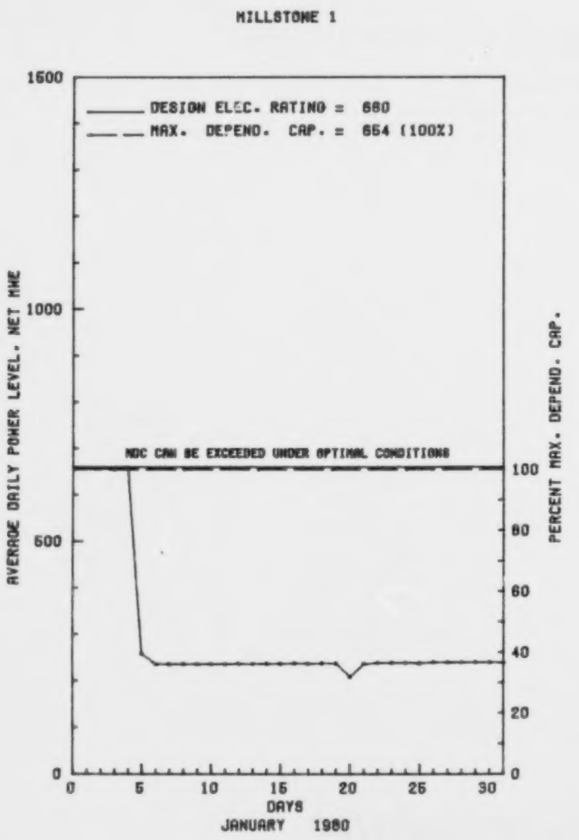
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-035/ 03L	12/04/79	12/20/79	30 Day LER	One of four main steam line low pressure switches found with a setpoint three psi under T.S. limits
79-036/ 03L	12/19/79	01/18/80	30 Day LER	Water hammer experienced in isolation condenser steam line following a reactor scram

REVIEWED BY OMPA LOEB: John J. Riggs **DATE:** 3-19-80

REVIEWED BY NRR DOR: S. J. Nowicki **DATE:** 3-20-80

MILLSTONE 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-245
2. Utility Contact: R. Young (203) 447-1792 x475
3. Licensed Thermal Power (MWt): 2011
4. Nameplate Rating (Gross MWe): 735 x 0.9 = 662
5. Design Electrical Rating (Net MWe): 660
6. Maximum Dependable Capacity (Gross MWe): 684
7. Maximum Dependable Capacity (Net MWe): 654
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): 262
10. Reasons for Restrictions, If Any:
ISOLATION CONDENSER OUT OF SERVICE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	744.0	86,424.0
12. Hours Reactor Was Critical	744.0	744.0	61,129.0
13. Reactor Reserve Shutdown Hours	.0	.0	935.0
14. Hours Generator On-Line	744.0	744.0	58,839.9
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	709,452	709,452	104,757,397
17. Gross Electrical Energy (MWH)	229,900	229,900	35,461,296
18. Net Electrical Energy (MWH)	216,865	216,865	33,856,647
19. Unit Service Factor	100.0	100.0	73.2
20. Unit Availability Factor	100.0	100.0	73.2
21. Unit Capacity Factor (MDC Net)	44.6	44.6	64.4
22. Unit Capacity Factor (DER Net)	44.2	44.2	63.8
23. Unit Forced Outage Rate	.0	.0	16.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
01	01/05/80	F	648.0	A	5	80-03/3L	CE	HTEXCH	REDUCED POWER AND RESTRICTED TO 40% DUE TO THE ISOLATION CONDENSER OUT OF SERVICE. THE UNIT WILL BE RETURNED TO FULL POWER UPON RESOLVEMENT OF CONDENSER SUPPORT MODIFICATIONS.

SUMMARY

OPERATION WAS ROUTINE AT A 40% POWER RESTRICTION WITH THE ISOLATION CONDENSER OUT OF SERVICE FROM 01/05/80 THROUGH THE END OF THE REPORT PERIOD. THE UNIT WILL RETURN TO FULL POWER FOLLOWING CONDENSER SUPPORT MODIFICATIONS.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

MILLSTONE 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Connecticut/New London 5 miles SW of New London
2.	Electric Reliability Council	NPCC
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2700
5.	Design Electrical Rating (MWE Net)	870
6.	Date of Initial Criticality	October 17, 1975
7.	Date of Commercial Operation	December 26, 1975
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Long Island Sound

Utility & Contractor Information

10.	Licensee	Northeast Nuclear Energy Company
11.	Corporate Address	P. O. Box 270 Hartford, Connecticut
12.	Corporate Contact	W. G. Counsil, Vice President
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Combustion Engineering
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	IE Senior Inspector	E. McCabe (FTS: 244-2000)
19.	IE Resident Inspector	T. Shedlosky (203) 447-3179
20.	Licensing Project Manager	E. L. Conner, Jr.
21.	Docket Number	50-336
22.	License Number/Date of Issue	DPR-65, 8/1/75
23.	Public Document Room	Waterford Public Library Rope Ferry Road Route 156 Waterford, Connecticut 06385

INSPECTION STATUS

ENFORCEMENT STATUS

ON 11/7/79, AN OPEN CAN OF COMBUSTIBLE DEBRIS WAS OBSERVED IN THE UNIT 1 CONTROL ROOM, AND AN OPEN CAN OF COMBUSTIBLE DEBRIS WAS OBSERVED IN THE UNIT 2 D.C. SWITCHGEAR ROOM. EVIDENCE OF SMOKING WAS OBSERVED IN THE UNIT 1 CONTROL ROOM MAIN CONTROL BOARD.
(7927 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

None.

FACILITY ITEMS (PLANS AND PROCEDURES):

None

MANAGERIAL ITEMS:

None

PLANT STATUS:

- The unit operated at a nominal 100% power during the month.
- LAST IE SITE INSPECTION DATE: 1/31/80
- INSPECTION REPORT NO: 50-336/80-01

SUMMARY

INSPECTION SUMMARY

- 50-336/79-26 - Nov 3-6: Special, announced inspection by one region-based inspector (20 hours) of the re-examination of defects in the feedwater piping. Noncompliances: None.
- 50-336/79-27 - Nov 5-9: Routine unannounced inspection by one region-based inspector of the Fire Protection/Prevention Program (28 hours) including: assignment of functional responsibilities; implementation of administrative controls; fire brigade training; ignition source and combustible materials control; fire protection systems and components; plant modifications; surveillance test records; and critical fire areas. Noncompliances: One (Failure to follow housekeeping procedures).
- 50-336/79-28 - Nov 27-30 and Dec 3-7: Routine, unannounced inspection (49 hours) by a region-based inspector and an NRR engineer of the Quality Assurance Program (QAP) implementation in the areas of: QA/QC administration; design changes/modifications; document control; QAP changes; and followup on previously identified items. Noncompliances: None.
- 50-336/79-31 - Dec 19-20: Special, unannounced inspection by a region-based inspector of the licensee's response to Bulletin 79-19 (7 hours) including: regulatory requirements, procedures, training, audits, records of shipments and onsite observations. Noncompliances: None.

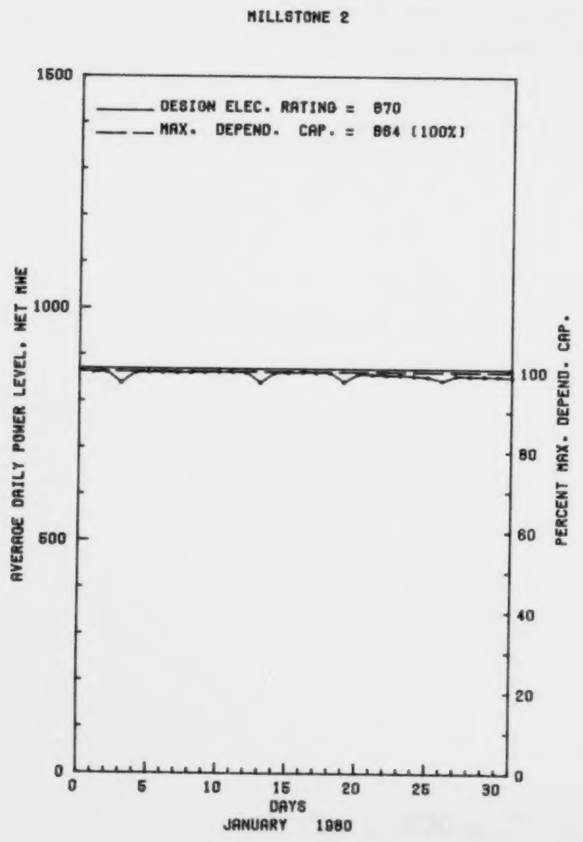
REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-039/03L	11/28/79	12/26/79	30 Day LER	During maintenance on several containment isolation valves, local leak rate tests were not performed as required by Technical Specifications
79-040/03L	12/04/79	01/03/80	30 Day LER	CST level dropped below T.S. limit due to filling and draining of steam generators
79-041/03L	12/28/79	01/23/80	30 Day LER	HPCI injection valves over traveled 3 percent during routine surveillance

REVIEWED BY OMPA LOEB: MK Beebe **DATE:** 03/19/80
REVIEWED BY NRR DOR: E L Conner, Jr. **DATE:** 3/20/80

MILLSTONE 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

ITEMS 21 AND 22 ARE CALCULATED BY USING A WEIGHTED CAPACITY FACTOR

1. Docket Number: 50-336
2. Utility Contact: G. H. Howlett (203) 447-1791 x364
3. Licensed Thermal Power (MWt): 2700
4. Nameplate Rating (Gross MWe): 1011 x 0.9 = 910
5. Design Electrical Rating (Net MWe): 870
6. Maximum Dependable Capacity (Gross MWe): 895
7. Maximum Dependable Capacity (Net MWe): 864
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	744.0	35,952.0
12. Hours Reactor Was Critical	744.0	744.0	25,822.7
13. Reactor Reserve Shutdown Hours	.0	.0	2,072.4
14. Hours Generator On-Line	744.0	744.0	24,564.2
15. Unit Reserve Shutdown Hours	.0	.0	335.4
16. Gross Thermal Energy (MWH)	2,001,005	2,001,005	59,699,633
17. Gross Electrical Energy (MWH)	660,962	660,962	19,245,607
18. Net Electrical Energy (MWH)	637,711	637,711	18,417,969
19. Unit Service Factor	100.0	100.0	68.3
20. Unit Availability Factor	100.0	100.0	69.3
*21. Unit Capacity Factor (MDC Net)	99.2	99.2	62.7
22. Unit Capacity Factor (DER Net)	98.5	98.5	61.3
23. Unit Forced Outage Rate	.0	.0	23.3

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUEL, JULY 1, 1980, 9 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWN/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	None								

SUMMARY

OPERATION WAS AT NEAR FULL POWER DURING THE REPORT PERIOD, WITH NO REDUCTIONS OR OUTAGES REPORTED.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

MONTICELLO

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Minnesota/Wright 30 miles NW of Minneapolis
2.	Electric Reliability Council	MARCA
3.	Type of Reactor	BWR
4.	Power Level (MWT)	1670
5.	Design Electrical Rating (MWE Net)	545
6.	Date of Initial Criticality	December 10, 1970
7.	Date of Commercial Operation	June 30, 1971
8.	Condenser Cooling Method	Once Through with Cooling Tower
9.	Condenser Cooling Water Source	Mississippi River

Utility & Contractor Information

10.	Licensee	Northern States Power Co.
11.	Corporate Address	414 Nicollet Mall - 8th Floor Minneapolis, Minnesota 55401
12.	Corporate Contact	G. H. Neils (Oper.) L. O. Mayer (Lic.)
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	GE
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heisman
18.	IE Senior Inspector	R. F. Warnick
19.	IE Principal Inspector	N. C. Choules
20.	Licensing Project Manager	T. Kevern
21.	Docket Number	50-263
22.	License Number/Date of Issue	DPR-22 issued 9/8/70
23.	Public Document Room	Environmental Conservation Library Minneapolis Public Library 300 Nicollet Mall Minneapolis, Minnesota 55401

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

FACILITY ITEMS(PLANS AND PROCEDURES):

- The licensee is building an office building extension. The radiation protection group will have new offices and will relocate mask fitting and whole body counting equipment to a room adjacent to these new offices. (79-19)

PLANT STATUS

- The plant continued in the coastdown mode of operation. Power level at the end of the month was 79%. A refueling outage is scheduled to begin February 22, 1980.

+LAST IE SITE INSPECTION DATE: December 17-18 and 21, 1979

- INSPECTION REPORT NO: 79-21

SUMMARY

INSPECTION SUMMARY

- Inspection on December 17-18, (79-19): Nonroutine, announced inspection of the licensee's response to IE Bulletin No. 79-19. The inspection involved 12 inspector hours onsite by one NRC inspector. No items of noncompliance or deviations were identified.
- Inspection on December 17-18 and 21, (79-21): Routine, announced inspection of confirmatory measurements, including the licensee's program for quality control of analytical measurements; evaluation of the licensee's analytical results of plant effluent samples obtained during a previous inspection; collection of plant effluent samples for future comparative analyses; and licensee internal audit. The inspection involved 12 inspector hours onsite by one NRC inspector. No apparent items of noncompliance or deviations were identified for the areas inspected.

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-21/ 03L-0	11/19/79	12/19/79	30 Day LER	Anchor bolts for Snubber SS-31 found inadequate.
79-22/ 03L-0	12/13/79	01/11/80	30 Day LER	Small condensate line leak.
79-23/ 01T-0 update 01/14/80	12/14/79	12/28/79	14 Day LER	MAPLHGR exposure oversight.
79-24/ 03L-0	12/20/79	01/18/80	30 Day LER	RCIC steam drain line steam leak.

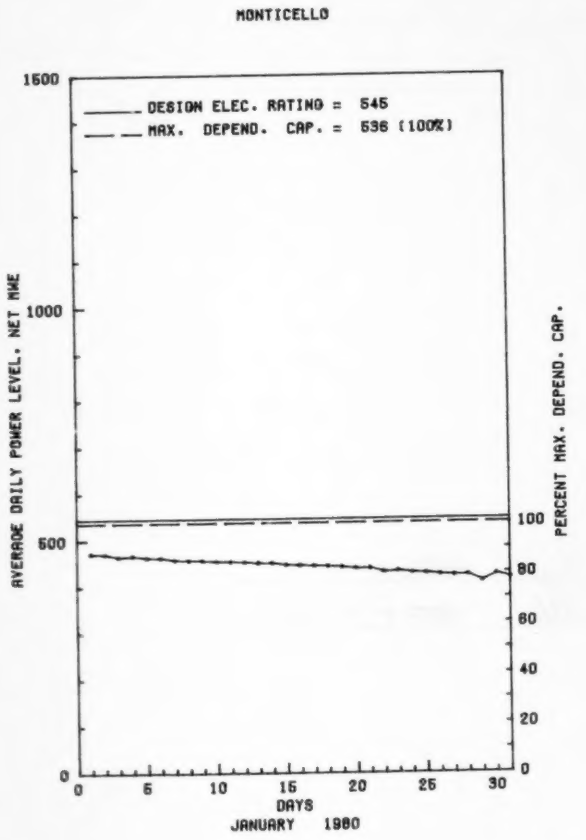
==

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 3-19-80

REVIEWED BY NRR DOR: **DATE:** 3/20/80

MONTICELLO

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-263
2. Utility Contact: A. L. Myrabo (612) 295-5151
3. Licensed Thermal Power (MWt): 1670
4. Nameplate Rating (Gross MWe): 632 x 0.9 = 569
5. Design Electrical Rating (Net MWe): 545
6. Maximum Dependable Capacity (Gross MWe): 564
7. Maximum Dependable Capacity (Net MWe): 536
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	744.0	75,289.0
12. Hours Reactor Was Critical	744.0	744.0	62,274.9
13. Reactor Reserve Shutdown Hours	.0	.0	950.7
14. Hours Generator On-Line	744.0	744.0	60,716.1
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,023,365	1,023,365	99,966,227
17. Gross Electrical Energy (MWH)	345,350	345,350	31,323,410
18. Net Electrical Energy (MWH)	330,326	330,326	29,962,530
19. Unit Service Factor	100.0	100.0	80.6
20. Unit Availability Factor	100.0	100.0	80.6
21. Unit Capacity Factor (MDC Net)	82.8	82.8	74.2
22. Unit Capacity Factor (DER Net)	81.5	81.5	73.0
23. Unit Forced Outage Rate	.0	.0	6.4

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
REFUELING - FEBRUARY 22, 1980 - 40 DAYS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
NONE									

SUMMARY

OPERATION WAS ROUTINE THROUGHOUT THE REPORT PERIOD IN AN END OF CYCLE COASTDOWN.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

NINE MILE POINT 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	New York/Oswego 8 miles NE of Oswego
2.	Electric Reliability Council	NPCC
3.	Type of Reactor	BWR
4.	Power Level (MWT)	1850
5.	Design Electrical Rating (MWE Net)	620
6.	Date of Initial Criticality	September 5, 1969
7.	Date of Commercial Operation	December 1969
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Lake Ontario

Utility & Contractor Information

10.	Licensee	Niagara Mohawk Power Corporation
11.	Corporate Address	300 Erie Boulevard West Syracuse, New York 13202
12.	Corporate Contact	R. Schneider, Vice President Electric Operations
13.	Architect/Engineer	Utility
14.	Nuclear Steam Supply System	GE
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Senior Inspector	E. McCabe
19.	IE Principal Inspector	W. Baunack
20.	Licensing Project Manager	P. Polk
21.	Docket Number	50-220
22.	License Number/Date of Issue	DPR-63, 12/26/74
23.	Public Document Room	State University College of Oswego Penfield Library - Documents Oswego, NY 13126 (315)341-2323

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEM AND COMPONENT PROBLEMS:

None

FACILITY ITEMS (PLANS AND PROCEDURES):

+ A 3 to 4 week shutdown is planned, starting 3/1, for TMI Short Term Lessons Learned Modifications and other maintenance.

MANAGERIAL ITEMS:

None

PLANT STATUS:

+ The unit operated continuously during the month of January. Representative stack and offgas release rates are 13 and 13,300 microcuries per second, respectively.

+ LAST IE SITE INSPECTION DATE: 2/4-8/80

+ INSPECTION REPORT NO: 50-220/80-01

SUMMARY

INSPECTION SUMMARY

No inspections completed during this report period.

REPORTS RECEIVED FROM LICENSEE

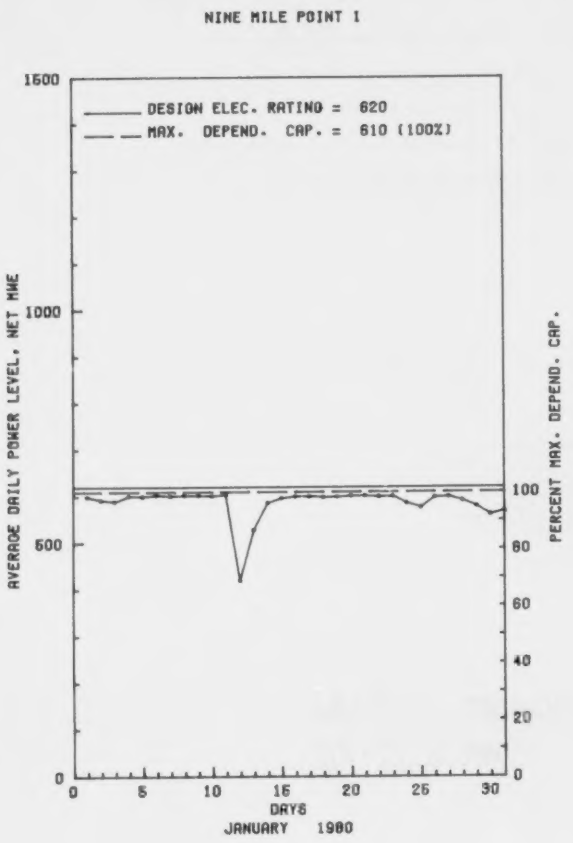
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
80-001/ 03L	01/02/80	01/10/80	30 Day LER	Average linear heat generator rate greater than allowed by TS
80-002/ 04L	01/09/80	01/16/80	Environmental Deviation	Condenser inlet screenwell discharge temp greater than EST limits due to faulty transmitter
80-003/ 01T	01/09/80	01/21/80	14 Day LER	Failure to conduct surveillance test at required interval

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 3-19-80

REVIEWED BY NRR DOR: Philip J. Polk **DATE:** 3-21-80

NINE MILE POINT 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-220
2. Utility Contact: B. Hawksley (315) 343-2110 x1312
3. Licensed Thermal Power (MWt): 1850
4. Nameplate Rating (Gross MWe): 755 x 0.85 = 642
5. Design Electrical Rating (Net MWe): 620
6. Maximum Dependable Capacity (Gross MWe): 630
7. Maximum Dependable Capacity (Net MWe): 610
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	744.0	89,856.0
12. Hours Reactor Was Critical	744.0	744.0	66,096.7
13. Reactor Reserve Shutdown Hours	.0	.0	1,204.2
14. Hours Generator On-Line	744.0	744.0	63,554.5
15. Unit Reserve Shutdown Hours	.0	.0	20.2
16. Gross Thermal Energy (MWH)	1,327,272	1,327,272	103,068,674
17. Gross Electrical Energy (MWH)	450,983	450,983	33,960,493
18. Net Electrical Energy (MWH)	436,353	436,353	32,886,118
19. Unit Service Factor	100.0	100.0	70.7
20. Unit Availability Factor	100.0	100.0	70.8
21. Unit Capacity Factor (MDC Net)	95.1	96.1	60.0
22. Unit Capacity Factor (DER Net)	94.6	94.6	59.0
23. Unit Forced Outage Rate	.0	.0	9.3

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	01/07/80	S	2.5	H	5				TO PULL SHAPPING RODS
	01/12/80	S	5.3	H	5				ROD SWAP.
	01/24/80	F	2.3	H	5				INTAKE TUNNEL ICE - REDUCED LOAD TO REVERSE FLOW.
	01/25/80	S	1.0	H	5				REDUCED LOAD TO CHANGE INTAKE TUNNEL TO NORMAL FLOW PATH.
	01/30/80	F	2.0	H	5				REDUCED LOAD - INTAKE TUNNEL ICED UP.
	01/31/80	S	1.0	H	5				CHANGED INTAKE TUNNEL FLOW TO NORMAL PATH.

SUMMARY

OPERATION WAS AT NEAR FULL POWER DURING THE REPORT PERIOD, WITH BRIEF REDUCTIONS FOR CONTROL ROD ADJUSTMENTS AND CONDENSER CIRCULATING WATER REVERSE FLOW OPERATIONS. THE NO. 15 REACTOR RECIRCULATION PUMP WAS OUT OF SERVICE AND ISOLATED DUE TO MECHANICAL PROBLEMS THROUGHOUT THE REPORT PERIOD.

LEGEND

[1] Type:
F-Forced
S-Scheduled

[2] Reason:
A-Equipment Failure
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error
H-Other

[3] Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Continuations
5-Load Reduction
9-Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

NORTH ANNA 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Louisa County, Virginia 40 Miles NW of Richmond, Virginia
2.	Electric Reliability Council	Southeastern
3.	Type of Reactor	PWR
4.	Licensee Power Level (MWT)	2775
5.	Design Electrical Rating (MWE Net)	907
6.	Date of Initial Criticality	April 5, 1978
7.	Date of Commercial Operation	June 6, 1978
8.	Condenser Cooling Method	Once through
9.	Condenser Cooling Water Source	Lake Anna

Utility & Contractor Information

10.	Licensee	Virginia Electric & Power Co.
11.	Corporate Address	P. O. Box 26666 Richmond, Virginia 23261
12.	Corporate Contact	J. H. Ferguson Exec. V.P. - Power
13.	Architect/Engineer	Stone & Webster
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. Lewis
18.	IE Senior Inspector	P. J. Kellog
19.	IE Resident Inspector	M. S. Kidd
20.	Licensing Project Manager	L. Engle
21.	Docket Number	50-338
22.	License Number/Date of Issue	NPF-4, 11/26/77
23.	Public Document Room	Alderman Library/Manuscripts Dept. Univ. of Virginia/Charlottesville VA 22901 & Louisa County Courthouse, Louisa, VA 23093

INSPECTION STATUS

ENFORCEMENT STATUS

AS REQUIRED BY TECHNICAL SPECIFICATION 6.9.1.9, WRITTEN REPORTS OF THE TYPES OF EVENTS LISTED IN THAT SPECIFICATION SHALL BE SUBMITTED TO THE DIRECTOR OF THE NRC REGIONAL OFFICE WITHIN 30 DAYS OF THE OCCURRENCE OF THE EVENT. SIMILARLY, SPECIFICATION 6.9.1.8 REQUIRES THAT WRITTEN REPORTS OF EVENTS OF THE TYPES IN THAT SPECIFICATION BE SUBMITTED WITHIN 14 DAYS OF THE EVENT OCCURRENCE: (A) CONTRARY TO THE ABOVE, LICENSEE EVENT REPORTS (LERS) NUMBERS 79-123, 79-124, 79-129, AND 79-134, ALL REPORTING EVENTS AS LISTED IN SPECIFICATION 6.9.1.9 WERE SUBMITTED TO THE DIRECTOR NRC REGIONAL OFFICE BETWEEN 31 TO 33 DAYS AFTER THE EVENT OCCURRENCE; (B) CONTRARY TO THE ABOVE, LER NUMBER 79-141, REPORTING AN EVENT AS LISTED IN SPECIFICATION 6.9.1.8 WAS SUBMITTED TO THE DIRECTOR NRC REGIONAL OFFICE 22 DAYS AFTER THE EVENT OCCURRENCE.
(7950 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:
ROUTINE OPERATIONS AFTER COMPLETION OF REFUELING OUTAGE ON 1/22/30.

LAST IE SITE INSPECTION DATE: FEBRUARY 8, 1980

INSPECTION REPORT NO: 50-338/80-08

SUMMARY

INSPECTION SUMMARY

INSPECTION NOVEMBER 5-13 AND 26-30 (79-48): THIS ROUTINE INSPECTION BY THE RESIDENT INSPECTOR INVOLVED EIGHT HOURS ON SITE IN THE AREAS OF IE CICULARS AND NOTICES AND LICENSEE EVENT REPORTS. NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION DECEMBER 17-21 (79-49): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 32 INSPECTOR-HOURS ON SITE IN THE AREAS OF CALIBRATION OF IN-PLANT INSTRUMENTATION REQUIRED BY TECHNICAL SPECIFICATIONS; CALIBRATION OF MEASURING AND TEST EQUIPMENT USED ON SAFETY-RELATED SYSTEMS; AND SURVEILLANCE OF IN-PLANT EQUIPMENT REQUIRED BY TECHNICAL SPECIFICATIONS. OF THE THREE AREAS INSPECTED, FOUR APPARENT ITEMS OF NONCOMPLIANCE WERE FOUND IN THREE AREAS (INFRACTION - FAILURE TO IMPLEMENT SURVEILLANCE TEST; INFRACTION - FAILURE TO CONDUCT PROPER REVIEW OF SURVEILLANCE TEST RESULTS; DEFICIENCY - FAILURE TO MAINTAIN RECORDS FOR TEST AND MEASURING EQUIPMENT; AND INFRACTION - FAILURE TO FOLLOW SURVEILLANCE TEST PROCEDURES).

INSPECTION DECEMBER 10-12 (79-50): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 16 INSPECTOR-HOURS ON SITE IN THE AREAS OF PREVIOUS IDENTIFIED CONCERNS, LICENSEE COMMITMENTS FOR THE REFUELING OUTAGE, AND LICENSEE EVENT REPORTS. OF THE THREE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN TWO AREAS; ONE ITEM OF NONCOMPLIANCE WAS FOUND IN ONE AREA (INFRACTION - FAILURE TO FORWARD WRITTEN REPORTS OF LICENSEE EVENTS IN SPECIFIED TIMEFRAME).

INSPECTION DECEMBER 18-20 (79-51): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 22 INSPECTOR-HOURS ON SITE IN THE AREAS RELATED TO DIESEL GENERATORS AND CHARGING PUMPS. OF THE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION DECEMBER 3-21 (79-52): THIS ROUTINE INSPECTION BY THE RESIDENT INSPECTOR INVOLVED 23 HOURS ON SITE IN THE AREAS OF PREVIOUSLY IDENTIFIED OPEN ITEMS, GENERAL EMPLOYEE TRAINING, EMERGENCY PROCEDURE TRAINING, PLANT TOURS, LICENSEE EVENT REPORTS, AND PLANT STATUS RELATIVE TO STARTUP FROM THE REFUELING OUTAGE NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

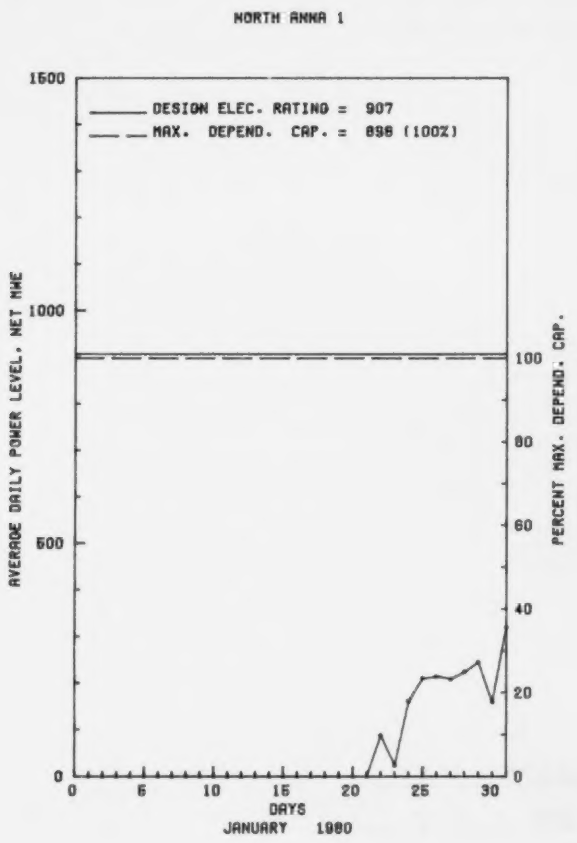
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-148/ 01T-0	12/18/79	12/13/79	14 DAY LER	OVERSTRESS DUE TO USE OF INCORRECT VALVE WEIGHTS ON COMPONENT COOLING WATER PIPING.
79-151/ 01T-0	11/05/79	11/20/79	14 DAY LER	SEISMIC DESIGN FOR AUXILIARY SYSTEMS IN ERROR DUE TO ERRORS IN USE OF RESPONSE SPECTRUM CURVES BY ARCHE-TECT-ENGINEER.
79-152/ 0TT-0	11/19/79	12/04/79	14 DAY LER	WESTINGHOUSE REPORTS NONCONSERVATISM IN DROPPED ROD ANALYSIS FOR HIGH NEGATIVE POWER RANGE.
79-154/ 03L-0	11/29/79	12/27/79	30 DAY LER	STEAM GENERATOR SUPPORT TEMPERATURE DETECTOR FOUND OUT OF CALIBRATION.
79-155/ 03L-0	12/03/79	12/27/79	30 DAY LER	TWO PRESSURIZER SAFETY VALVES TESTED BELOW ACCEPTABLE RELIEF SETPOINT.
79-158/ 01T-0	12/12/79	12/20/79	14 DAY LER	PRESSURE DIFFERENTIAL SWITCHES ON SERVICE WATER TO CONTROL ROOM AIR CONDITIONING NOT SEISMICALLY QUALIFIED.

REVIEWED BY OMPA LOEB: T. C. CINTULA **DATE:** 3·19·80

REVIEWED BY NRR DOR: **DATE:** 3·19·80

NORTH ANNA 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-338
2. Utility Contact: W. R. Madison (703) 894-5151
3. Licensed Thermal Power (MWt): 2775
4. Nameplate Rating (Gross MWe): 1089 x 0.9 = 980
5. Design Electrical Rating (Net MWe): 907
6. Maximum Dependable Capacity (Gross MWe): 928
7. Maximum Dependable Capacity (Net MWe): 898
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	744.0	14,521.0
12. Hours Reactor Was Critical	343.4	343.4	10,508.6
13. Reactor Reserve Shutdown Hours	30.0	30.0	178.3
14. Hours Generator On-Line	202.8	202.8	10,255.9
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	206,696	206,696	26,486,609
17. Gross Electrical Energy (MWH)	52,764	52,764	8,399,113
18. Net Electrical Energy (MWH)	44,468	44,468	7,897,914
19. Unit Service Factor	27.3	27.3	70.6
20. Unit Availability Factor	27.3	27.3	70.6
21. Unit Capacity Factor (MDC Net)	6.7	6.7	60.6
22. Unit Capacity Factor (DER Net)	6.6	6.6	60.0
23. Unit Forced Outage Rate	4.0	4.0	3.3

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
79-16	09/25/79	S	510.9	C	4		RC	FUELXX	REFUELING OUTAGE CONTINUES.
80-01	01/16/80	F	.0	A	5				HIGH VIBRATION "A" REACTOR COOLANT PUMP. TIGHTENED 1 COUPLING NUT.
80-02	01/17/80	F	.0	A	5				BALANCE "A" REACTOR COOLANT PUMP.
80-03	01/17/80	F	.0	A	5				BALANCE "A" REACTOR COOLANT PUMP.
80-04	01/18/80	F	.0	G	5				VOLTAGE SPIKE DURING REACTOR PROTECTION PERIODIC TEST CAUSE A LOSS OF P-7 CAUSING REACTOR TRIP.
80-05	01/19/80	S	.0	B	5				SCHEDULED TEST TRIP DURING LOW POWER PHYSICS TESTING.
80-06	01/22/80	S	21.9	B	1				SCHEDULED SHUTDOWN FOR TURBINE OVERSPEED TEST AND MAINTENANCE ON NUCLEAR INSTRUMENTATION.
80-07	01/29/80	F	8.4	H	3				LOSS OF FEEDWATER DUE TO CLOGGED CONDENSATE PUMP SUCTION STRAINERS. CLEANED THE STRAINERS.

SUMMARY

THE UNIT CAME ON LINE ON 01/22/80 FROM REFUELING. LOAD REDUCTIONS WERE NECESSARY FOR TESTING AND MAINTENANCE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

OCONEE 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	South Carolina/Oconee 30 miles W of Greenville
2.	Electric Reliability Council	Southeastern
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2568
5.	Design Electrical Rating (MWE Net)	887
6.	Date of Initial Criticality	April 19, 1973
7.	Date of Commercial Operation	July 15, 1973
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Keowee Lake

Utility & Contractor Information

10.	Licensee	Duke Power Company
11.	Corporate Address	422 S. Church Street Charlotte, North Carolina 28242
12.	Corporate Contact	W. O. Parker, Jr.
13.	Architect/Engineer	DPC and Bechtel (Containment Bldg.)
14.	Nuclear Steam Supply System	B&W
15.	Constructor	DPC

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R.C. Lewis
18.	IE Senior Inspector	E.H. Verdery
19.	IE Resident Inspector	F. Jape
20.	Licensing Project Manager	G. Rivenbark
21.	Docket Number	50-269
22.	License Number/Date of Issue	DPR-38, 2/6/73
23.	Public Document Room	Oconee County Library 201 S. Spring Street Walhalla, South Carolina 29691

INSPECTION STATUS

ENFORCEMENT STATUS

AS REQUIRED BY 10 CFR 50 APPENDIX "B" CRITERION V AS IMPLEMENTED BY DUKE POWER COMPANY TOPICAL REPORT 1A SECTION 17.2.5 AND OCONEE TECHNICAL SPECIFICATION SECTION 6.4.1 ACTIVITIES AFFECTING QUALITY SHALL BE PRESCRIBED BY DOCUMENTED INSTRUCTIONS, PROCEDURES OR DRAWINGS ... AND SHALL BE ACCOMPLISHED IN ACCORDANCE WITH THESE INSTRUCTIONS PROCEDURES OR DRAWINGS: (1) DUKE POWER COMPANY PROCEDURE MP/0/A/3019/01 PARAGRAPH 11.3 NOTE 3 REQUIRES THAT 'ANY ATTACHMENTS (EXTRA PIPES, CABLE TRAYS, EXTRA STEEL, ETC.) MUST BE SHOWN ON THE SUPPORT RESTRAINT DESIGN DRAWING. CONTRARY TO THE ABOVE, HANGER NUMBERS 51A-0-479A-H21C, 51A-0-479A-H18C AND 2-53B-2-0-436E-R3 HAD ATTACHMENTS THAT WERE NOT SHOWN ON THE SURVEILLANCE "AS-BUILT" DESIGN DRAWINGS. (2) DUKE POWER COMPANY PROCEDURE SPECIFICATION NO. OS-0020.00-00-003 PARAGRAPH 4.4.2 REQUIRES THAT WEDGE ANCHORS SHALL HAVE THE PROPER TEST TORQUE APPLIED TO ENSURE EXPANSION OF THE ANCHOR. CONTRARY TO THE ABOVE, HANGER 1-07A-400B-DE014 PLATE A WAS FOUND TO HAVE THREE WEDGE ANCHORS THAT HAD BEEN TORQUED TO LESS THAN THE REQUIRED TEST TORQUE VALUE.
(7939 INFRACTION)

FAILURE TO FOLLOW 10 CFR 73.55(D) REQUIREMENTS.
(7939 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

REFUELING OUTAGE.

LAST IE SITE INSPECTION DATE: JANUARY 2-31, 1980

INSPECTION REPORT NO: 50-269/80-05

SUMMARY

INSPECTION SUMMARY

INSPECTION NOVEMBER 26-27 AND DECEMBER 4-5 (79-34): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 10.5 INSPECTOR-HOURS ON SITE IN THE AREAS OF HIGH PRESSURE INJECTION MODIFICATION - WELDING, AND RECORDS REVIEW, UNIT 1; UNIT 1 FIFTH OUTAGE ISI INSPECTION, WORK OBSERVATION AND REVIEW OF THE ISI PROGRAM, PROCEDURES AND RECORDS. OF THE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION NOVEMBER 27-30 (79-35): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 11 INSPECTOR-HOURS ON SITE IN THE AREAS OF RADIATION PROTECTION, INCLUDING QUALIFICATION OF HEALTH PHYSICS PERSONNEL, RADIOLOGICAL CONTROL PROCEDURES, POSTING, LABELING AND CONTROL; REVIEW OF LICENSEE ACTION ON I&E BULLETINS, CIRCULARS AND NOTICES AND FOLLOWUP ON AN ON SITE SPILL OF RADIOACTIVE WATER. OF THE FOUR AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN TWO AREAS; TWO APPARENT ITEMS OF NONCOMPLIANCE WERE FOUND IN TWO AREAS (INFRACTION - FAILURE TO FOLLOW PROCEDURE; AND DEFICIENCY - FAILURE TO LABEL CONTAINERS OF RADIOACTIVE MATERIAL).

INSPECTION NOVEMBER 1-30 (79-36): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 26 INSPECTOR-HOURS ON SITE IN THE AREAS OF PLANT OPERATIONS, LER REVIEW AND FOLLOWUP, REVIEW OF SELECTED EMERGENCY PROCEDURES, REVIEW OF DRAWING CONTROL AND REVIEW OF SELECTED TEST DATA. OF THE FIVE AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION DECEMBER 5-7 (79-38): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 8 INSPECTOR-HOURS ON SITE. THE AREAS INSPECTED WERE REFUELING ACTIVITIES IN UNIT 1, ROUTINE OPERATIONS OF UNIT 2 AND POST-REFUELING POWER ESCALATION TESTS OF UNIT 3. NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN THE THREE AREAS INSPECTED.

INSPECTION DECEMBER 3-5 (79-39): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 7 INSPECTOR-HOURS ON SITE IN THE AREAS OF PIPE SUPPORT BASEPLATE DESIGNS USING CONCRETE EXPANSION ANCHOR BOLTS WORK ACTIVITIES AND RECORDS; SEISMIC ANALYSIS FOR AS-BUILT SAFETY-RELATED PIPING SYSTEMS WORK ACTIVITIES AND RECORDS; SITE SECURITY SEARCH PROCEDURE ACTIVITIES. OF THE FIVE AREAS INSPECTED, TWO ITEMS OF NONCOMPLIANCE WERE FOUND IN FIVE AREAS.

MEETING NOVEMBER 30 (79-40): CORPORATE MANAGEMENT MEETING TO DISCUSS RESULTS OF ACTIONS REQUESTED BY LETTER, DATED OCTOBER 25, 1979, TO DUKE POWER COMPANY; ENFORCEMENT HISTORY, LICENSEE EVENTS AND THE LICENSEE'S TOTAL PERFORMANCE. LICENSEE MANAGEMENT ACKNOWLEDGED THE CONCERNS EXPRESSED BY NRC REPRESENTATIVES AND INDICATED THE CORRECTIVE ACTIONS PROGRAM OUTLINED IN THEIR AUDIT DATED NOVEMBER 16, 1979, SHOULD RESULT IN IMPROVED PERFORMANCE.

INSPECTION DECEMBER 3-20 (79-41): THIS ROUTINE, INSPECTION INVOLVED 18 INSPECTOR-HOURS ON SITE IN THE AREAS OF PLANT OPERATIONS, LICENSEE REPORTABLE EVENT, EMERGENCY POWER SYSTEM MAINTENANCE AND FOLLOWUP OF IEB 79-21. OF THE FOUR AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN THREE AREAS; ONE APPARENT ITEM OF NONCOMPLIANCE WAS FOUND IN ONE AREA (INFRACTION - FAILURE TO FOLLOW PROCEDURE RESULTING IN NONCOMPLIANCE WITH TECHNICAL SPECIFICATION 3.3).

INSPECTION JANUARY 9-11 (80-2): THIS ROUTINE ANNOUNCED INSPECTION INVOLVED 12 INSPECTOR-HOURS ON SITE IN THE AREAS OF PIPE SUPPORT BASEPLATE DESIGNS USING CONCRETE EXPANSION ANCHOR BOLTS; SEISMIC ANALYSIS FOR AS-BUILT SAFETY-RELATED PIPING. OF THE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-035/03L-0	12/05/79	01/04/80	30 DAY LER	INOPERABLE SPENT FUEL POOL (RIA-6) MONITOR.
79-036/03L-0	12/10/79	01/09/80	30 DAY LER	HIGH PRESSURE SERVICE WATER PUMP B INOPERABLE.

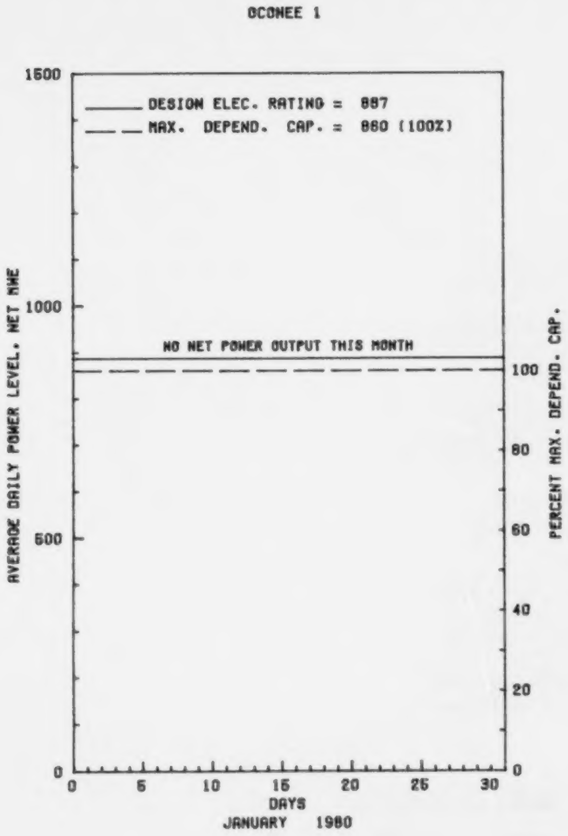
REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 3-19-80

REVIEWED BY NRR DOR: G W Rivenbark **DATE:** 3-21-80

GPO 871-667

OCONEE 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

CAPACITY FACTORS
CALCULATED WITH A
NEIGHTED AVERAGE

1. Docket Number: 50-269
2. Utility Contact: J. A. Reavis (704) 373-8552
3. Licensed Thermal Power (MWt): 2568
4. Nameplate Rating (Gross MWe): 1038 x 0.9 = 934
5. Design Electrical Rating (Net MWe): 887
6. Maximum Dependable Capacity (Gross MWe): 899
7. Maximum Dependable Capacity (Net MWe): 860
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	744.0	57,335.0
12. Hours Reactor Was Critical	.0	.0	40,514.7
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	.0	.0	37,947.1
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	0	89,218,119
17. Gross Electrical Energy (MWH)	0	0	30,915,300
18. Net Electrical Energy (MWH)	-2,316	-2,316	29,229,183
19. Unit Service Factor	.0	.0	66.1
20. Unit Availability Factor	.0	.0	66.1
21. Unit Capacity Factor (MDC Net)	.0	.0	59.0
22. Unit Capacity Factor (DER Net)	.0	.0	57.4
23. Unit Forced Outage Rate	.0	.0	18.0

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 02/27/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
1	11/21/79	S	374.0	C	4		RC	FUELXX	REFUELING COMPLETED.
2	01/16/80	S	370.0	D	9		XX	PIPEXX	IE BULLETIN 79-02 AND 79-14 (PIPE HANGER/SUPPORT INSPECTION AND MODIFICATION) CONTINUES.

SUMMARY

THE UNIT REMAINED SHUTDOWN THROUGHOUT THE REPORT PERIOD FOR REFUELING AND PIPE HANGER/SUPPORT INSPECTION AND MODIFICATION PER NRC BULLETINS 79-02 AND 79-14.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

OCONEE 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	South Carolina/Oconee 30 miles W of Greenville
2.	Electric Reliability Council	Southeastern
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2568
5.	Design Electrical Rating (MWE Net)	887
6.	Date of Initial Criticality	November 11, 1973
7.	Date of Commercial Operation	September 9, 1974
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Keowee Lake

Utility & Contractor Information

10.	Licensee	Duke Power Company
11.	Corporate Address	422 S. Church Street Charlotte, North Carolina 28242
12.	Corporate Contact	W. O. Parker, Jr.
13.	Architect/Engineer	DPC/Bechtel
14.	Nuclear Steam Supply System	B&W
15.	Constructor	DPC

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R.C. Lewis
18.	IE Senior Inspector	E.H. Verdery
19.	IE Resident Inspector	F. Jape
20.	Licensing Project Manager	G. Rivenbark
21.	Docket Number	50-270
22.	License Number/Date of Issue	DPR-47, 10/6/73
23.	Public Document Room	Oconee County Library 201 S. Spring Street Walhalla, South Carolina 29691

INSPECTION STATUS

ENFORCEMENT STATUS

AS REQUIRED BY 10 CFR 50 APPENDIX "B" CRITERION V AS IMPLEMENTED BY DUKE POWER COMPANY TOPICAL REPORT 1A SECTION 17.2.5 AND OCONEE TECHNICAL SPECIFICATION SECTION 6.4.1 ACTIVITIES AFFECTING QUALITY SHALL BE PRESCRIBED BY DOCUMENTED INSTRUCTIONS, PROCEDURES OR DRAWINGS ... AND SHALL BE ACCOMPLISHED IN ACCORDANCE WITH THESE INSTRUCTIONS PROCEDURES OR DRAWINGS: (1) DUKE POWER COMPANY PROCEDURE MP/0/A/3019/01 PARAGRAPH 11.5 NOTE 3 REQUIRES THAT 'ANY ATTACHMENTS (EXTRA PIPES, CABLE TRAYS, EXTRA STEEL, ETC.) MUST BE SHOWN ON THE SUPPORT RESTRAINT DESIGN DRAWING. CONTRARY TO THE ABOVE, HANGER NUMBERS 51A-0-479A-H21C, 51A-0-479A-H18C AND 2-53B-2-0-436E-R3 HAD ATTACHMENTS THAT WERE NOT SHOWN ON THE SURVEILLANCE "AS-BUILT" DESIGN DRAWINGS. (2) DUKE POWER COMPANY PROCEDURE SPECIFICATION NO. OS-0020.00-00-003 PARAGRAPH 4.4.2 REQUIRES THAT WEDGE ANCHORS SHALL HAVE THE PROPER TEST TORQUE APPLIED TO ENSURE EXPANSION OF THE ANCHOR. CONTRARY TO THE ABOVE, HANGER 1-074-400B-DE014 PLATE A WAS FOUND TO HAVE THREE WEDGE ANCHORS THAT HAD BEEN TORQUED TO LESS THAN THE REQUIRED TEST TORQUE VALUE.
(7936 INFRACTION)

FAILURE TO FOLLOW 10 CFR 73.55(D) REQUIREMENTS.
(7936 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

POWER OPERATION.

+ LAST IE SITE INSPECTION DATE: JANUARY 2-31, 1980

+ INSPECTION REPORT NO: 50-270/80-04

SUMMARY

INSPECTION SUMMARY

+ INSPECTION NOVEMBER 27-30 (79-32): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 10 INSPECTOR-HOURS ON SITE IN THE AREAS OF RADIATION PROTECTION, INCLUDING QUALIFICATION OF HEALTH PHYSICS PERSONNEL, RADIOLOGICAL CONTROL PROCEDURES, POSTING, LABELING AND CONTROL; REVIEW OF LICENSEE ACTION ON I&E BULLETINS, CIRCULARS AND NOTICES AND FOLLOWUP ON AN ON SITE SPILL OF RADIOACTIVE WATER. OF THE FOUR AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN TWO AREAS; TWO APPARENT ITEMS OF NONCOMPLIANCE WERE FOUND IN TWO AREAS (INFRACTION - FAILURE TO FOLLOW PROCEDURE; AND DEFICIENCY - FAILURE TO LABEL CONTAINERS OF RADIOACTIVE MATERIAL).

INSPECTION NOVEMBER 1-30 (79-33): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 26 INSPECTOR-HOURS ON SITE IN THE AREAS OF PLANT OPERATIONS, LER REVIEW AND FOLLOWUP, REVIEW OF SELECTED EMERGENCY PROCEDURES, REVIEW OF DRAWING CONTROL AND REVIEW OF SELECTED TEST DATA. OF THE FIVE AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-09/ 01T-0	12/09/79	12/21/79	14 DAY LER	HIGH PRESSURE INJECTION PUMP SUCTION VALVES INOPERABLE.

INSPECTION DECEMBER 5-7 (79-35): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 7 INSPECTOR-HOURS ON SITE. THE AREAS INSPECTED WERE REFUELING ACTIVITIES IN UNIT 1, ROUTINE OPERATIONS OF UNIT 2 AND POST-REFUELING POWER ESCALATION TESTS OF UNIT 3. NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN THE THREE AREAS INSPECTED.

INSPECTION DECEMBER 3-5 (79-36): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 7 INSPECTOR-HOURS ON SITE IN THE AREAS OF PIPE SUPPORT BASEPLATE DESIGNS USING CONCRETE EXPANSION ANCHOR BOLTS WORK ACTIVITIES AND RECORDS; SEISMIC ANALYSIS FOR AS-BUILT SAFETY-RELATED PIPING SYSTEMS WORK ACTIVITIES AND RECORDS; SITE SECURITY SEARCH PROCEDURE ACTIVITIES. OF THE FIVE AREAS INSPECTED, TWO ITEMS OF NONCOMPLIANCE WERE FOUND

MEETING NOVEMBER 30 (79-37): CORPORATE MANAGEMENT MEETING TO DISCUSS RESULTS OF ACTIONS REQUESTED BY LETTER, DATED OCTOBER 25, 1979, TO DUKE POWER COMPANY; ENFORCEMENT HISTORY, LICENSEE EVENTS AND THE LICENSEE'S TOTAL PERFORMANCE. LICENSEE MANAGEMENT ACKNOWLEDGED THE CONCERNS EXPRESSED BY NRC REPRESENTATIVES AND INDICATED THE CORRECTIVE ACTIONS PROGRAM OUTLINED IN THEIR AUDIT DATED NOVEMBER 16, 1979, SHOULD RESULT IN IMPROVED PERFORMANCE.

+ INSPECTION NOVEMBER 26-27 AND DECEMBER 4-5 (79-38): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 10 INSPECTOR-HOURS ON SITE IN THE AREAS OF HIGH PRESSURE INJECTION MODIFICATION - WELDING, AND RECORDS REVIEW, UNIT 1; UNIT 1 FIFTH OUTAGE ISI INSPECTION, WORK OBSERVATION AND REVIEW OF THE ISI PROGRAM, PROCEDURES AND RECORDS. OF THE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATION WERE IDENTIFIED.

INSPECTION DECEMBER 3-20 (79-39): THIS ROUTINE, INSPECTION INVOLVED 17 INSPECTOR-HOURS ON SITE IN THE AREAS OF PLANT OPERATIONS, LICENSEE REPORTABLE EVENT, EMERGENCY POWER SYSTEM MAINTENANCE AND FOLLOWUP OF IEB 79-21. OF THE FOUR AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN THREE AREAS; ONE APPARENT ITEM OF NONCOMPLIANCE WAS FOUND IN ONE AREA (INFRACTION - FAILURE TO FOLLOW PROCEDURE RESULTING IN NONCOMPLIANCE WITH TECHNICAL SPECIFICATION 3.3).

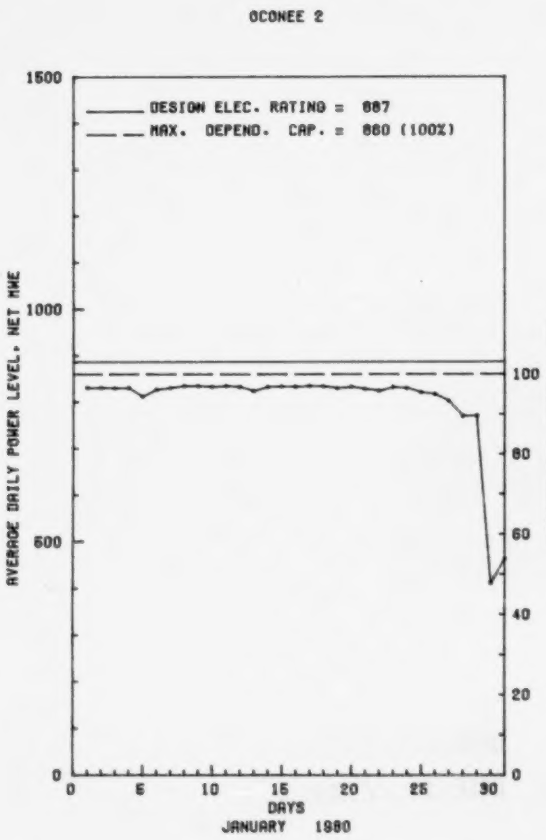
INSPECTION JANUARY 9-11 (80-1): THIS ROUTINE ANNOUNCED INSPECTION INVOLVED 12 INSPECTOR-HOURS ON SITE IN THE AREAS OF PIPE SUPPORT BASEPLATE DESIGNS USING CONCRETE EXPANSION ANCHOR BOLTS; SEISMIC ANALYSIS FOR AS-BUILT SAFETY-RELATED PIPING. OF THE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 3-19-80

REVIEWED BY NRR DOR: GW Rivenbark **DATE:** 3-21-80

OCONEE 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

CAPACITY FACTORS CALCULATED WITH A WEIGHTED AVERAGE

1. Docket Number: 50-270
2. Utility Contact: J. A. Reavis (704) 373-8552
3. Licensed Thermal Power (MWt): 2568
4. Nameplate Rating (Gross MWe): 1038 x 0.9 = 934
5. Design Electrical Rating (Net MWe): 887
6. Maximum Dependable Capacity (Gross MWe): 899
7. Maximum Dependable Capacity (Net MWe): 860
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	744.0	47,305.0
12. Hours Reactor Was Critical	743.2	743.2	34,339.2
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	731.7	731.7	33,507.8
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,845,969	1,845,969	79,893,373
17. Gross Electrical Energy (MWH)	622,140	622,140	27,136,496
18. Net Electrical Energy (MWH)	594,896	594,896	25,758,655
19. Unit Service Factor	98.3	98.3	70.8
20. Unit Availability Factor	98.3	98.3	70.8
*21. Unit Capacity Factor (MDC Net)	93.0	93.0	63.0
22. Unit Capacity Factor (DER Net)	90.1	90.1	61.5
23. Unit Forced Outage Rate	1.7	1.7	10.7

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUELING - MARCH 7, 1980 - 12 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
1	01/25/80	S	.0	H	5		RC	FUELXX	CONTROL ROD ADJUSTMENT AND POWER REDUCTION TO INCREASE CORE LIFE.
2	01/30/80	F	12.3	H	3		ZZ	RELAYX	AN ERROR IN RELAY TESTING IN THE 230 KV SUBSTATION RESULTED IN A UNIT TRIP.
3	01/31/80	F	.0	F	5		RC	FUELXX	HOLD IN POWER INCREASE DUE TO XENON AND CORE FLUX IMBALANCE.

SUMMARY

OPERATION WAS AT NEAR FULL POWER UNTIL 01/25/80, WHEN THE UNIT BEGAN AN END-OF-CYCLE COASTDOWN. ON 01/30/80, THE UNIT TRIPPED DUE TO AN ERROR IN RELAY TESTING.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

OCONEE 3

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	South Carolina/Oconee 30 miles W of Greenville
2.	Electric Reliability Council	Southeastern
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2568
5.	Design Electrical Rating (MWE Net)	887
6.	Date of Initial Criticality	September 5, 1974
7.	Date of Commercial Operation	December 16, 1974
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Keowee Lake

Utility & Contractor Information

10.	Licensee	Duke Power Company
11.	Corporate Address	422 S. Church Street Charlotte, North Carolina 28242
12.	Corporate Contact	W. O. Parker, Jr.
13.	Architect/Engineer	DPC/Bechtel
14.	Nuclear Steam Supply System	B&W
15.	Constructor	DPC

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R.C. Lewis
18.	IE Senior Inspector	E.H. Verdery
19.	IE Resident Inspector	F. Jape
20.	Licensing Project Manager	G. Rivenbark
21.	Docket Number	50-287
22.	License Number/Date of Issue	DPR-55, 7/19/74
23.	Public Document Room	Oconee County Library 201 S. Spring Street Walhalla, South Carolina 29691

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

100% POWER OPERATION.

* LAST IE SITE INSPECTION DATE: JANUARY 2-31, 1980

* INSPECTION REPORT NO: 50-287/80-04

SUMMARY

INSPECTION SUMMARY

* INSPECTION NOVEMBER 27-30 (79-35): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 10 INSPECTOR-HOURS ON SITE IN THE AREAS OF RADIATION PROTECTION, INCLUDING QUALIFICATION OF HEALTH PHYSICS PERSONNEL, RADIOLOGICAL CONTROL PROCEDURES, POSTING, LABELING AND CONTROL; REVIEW OF LICENSEE ACTION ON I&E BULLETINS, CIRCULARS AND NOTICES AND FOLLOWUP ON AN ON SITE SPILL OF RADIOACTIVE WATER. OF THE FOUR AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN TWO AREAS; TWO APPARENT ITEMS OF NONCOMPLIANCE WERE FOUND IN TWO AREAS (INFRACTION - FAILURE TO FOLLOW PROCEDURE; AND DEFICIENCY - FAILURE TO LABEL CONTAINERS OF RADIOACTIVE MATERIAL).

INSPECTION NOVEMBER 1-30 (79-36): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 25 INSPECTOR-HOURS ON SITE IN THE AREAS OF PLANT OPERATIONS, LER REVIEW AND FOLLOWUP, REVIEW OF SELECTED EMERGENCY PROCEDURES, REVIEW OF DRAWING CONTROL AND REVIEW OF SELECTED TEST DATA. OF THE FIVE AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION DECEMBER 5-7 (79-38): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 7 INSPECTOR-HOURS ON SITE. THE AREAS INSPECTED WERE REFUELING ACTIVITIES IN UNIT 1, ROUTINE OPERATIONS OF UNIT 2 AND POST-REFUELING POWER ESCALATION TESTS OF UNIT 3. NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN THE THREE AREAS INSPECTED.

MEETING NOVEMBER 30 (79-39): CORPORATE MANAGEMENT MEETING TO DISCUSS RESULTS OF ACTIONS REQUESTED BY LETTER, DATED OCTOBER 25, 1979, TO DUKE POWER COMPANY; ENFORCEMENT HISTORY, LICENSEE EVENTS AND THE LICENSEE'S TOTAL PERFORMANCE. LICENSEE MANAGEMENT ACKNOWLEDGED THE CONCERNS EXPRESSED BY NRC REPRESENTATIVES AND INDICATED THE CORRECTIVE ACTIONS PROGRAM OUTLINED IN THEIR AUDIT DATED NOVEMBER 16, 1979, SHOULD RESULT IN IMPROVED PERFORMANCE.

* INSPECTION NOVEMBER 26-27 AND DECEMBER 4-5 (79-40): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 10 INSPECTOR-HOURS ON SITE IN THE AREAS OF HIGH PRESSURE INJECTION MODIFICATION - WELDING, AND RECORDS REVIEW, UNIT 1; UNIT 1 FIFTH OUTAGE ISI INSPECTION, WORK OBSERVATION AND REVIEW OF THE ISI PROGRAM, PROCEDURES AND RECORDS. OF THE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION DECEMBER 3-20 (79-41): THIS ROUTINE, INSPECTION INVOLVED 17 INSPECTOR-HOURS ON SITE IN THE AREAS OF PLANT OPERATIONS, LICENSEE REPORTABLE EVENT, EMERGENCY POWER SYSTEM MAINTENANCE AND FOLLOWUP OF IEB 79-21. OF THE FOUR AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN THREE AREAS; ONE APPARENT ITEM OF NONCOMPLIANCE WAS FOUND IN ONE AREA (INFRACTION - FAILURE TO FOLLOW PROCEDURE RESULTING IN NONCOMPLIANCE WITH TECHNICAL SPECIFICATION 3.3).

INSPECTION JANUARY 9-11 (80-1): THIS ROUTINE ANNOUNCED INSPECTION INVOLVED 12 INSPECTOR-HOURS ON SITE IN THE AREAS OF PIPE SUPPORT BASEPLATE DESIGNS USING CONCRETE EXPANSION ANCHOR BOLTS; SEISMIC ANALYSIS FOR AS-BUILT SAFETY-RELATED PIPING. OF THE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

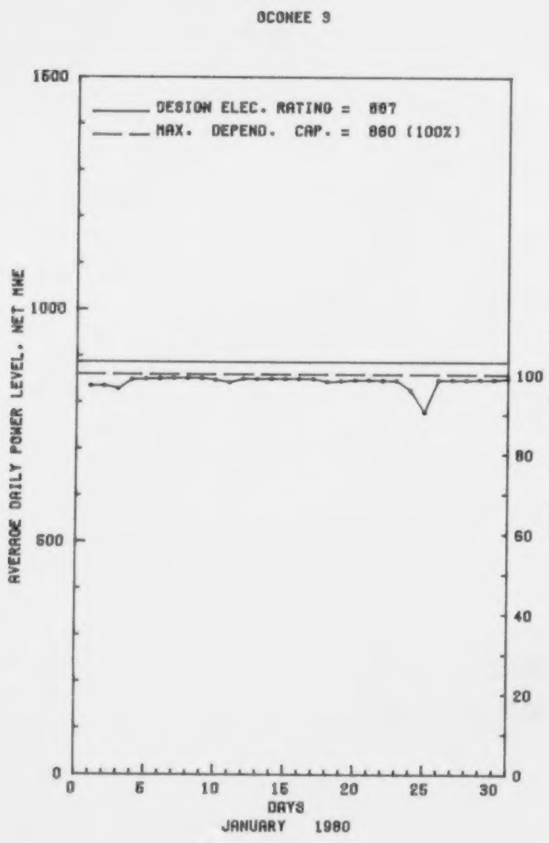
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
	NONE			

REVIEWED BY OMPA LOEB: John J. Ryzzo **DATE:** 3-19-80

REVIEWED BY NRR DOR: G W Rivenbark **DATE:** 3-21-80

OCONEE 3

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

CAPACITY FACTORS CALCULATED WITH A WEIGHTED AVERAGE

1. Docket Number: 50-287
2. Utility Contact: J. A. Reavis (704) 373-8552
3. Licensed Thermal Power (MWt): 2568
4. Nameplate Rating (Gross MWe): 1038 x 0.9 = 934
5. Design Electrical Rating (Net MWe): 887
6. Maximum Dependable Capacity (Gross MWe): 899
7. Maximum Dependable Capacity (Net MWe): 860
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	744.0	44,952.0
12. Hours Reactor Was Critical	744.0	744.0	32,637.9
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	744.0	744.0	31,805.9
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,896,002	1,896,002	76,277,607
17. Gross Electrical Energy (MWH)	655,270	655,270	26,406,534
18. Net Electrical Energy (MWH)	628,322	628,322	25,124,878
19. Unit Service Factor	100.0	100.0	70.8
20. Unit Availability Factor	100.0	100.0	70.8
21. Unit Capacity Factor (MDC Net)	98.2	98.2	64.7
22. Unit Capacity Factor (DER Net)	95.2	95.2	63.1
23. Unit Forced Outage Rate	.0	.0	17.8

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): RELIEF VALVE POSITION INDICATION - MARCH 15 - 2 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
1	01/02/80	F	.0	B	5		CH	PUMPXX	POWER REDUCED TO REMOVE 3E1 HEATER DRAIN PUMP FROM SERVICE FOR MAINTENANCE.
2	01/24/80	F	.0	B	5		CH	PUMPXX	3 "B" FEEDWATER PUMP TURBINE TRIPPED DURING NORMAL OIL PUMP TRIP TEST CAUSING RUN BACK ON UNIT LOAD.

SUMMARY

OPERATION WAS AT NEAR FULL POWER DURING THE REPORT PERIOD, WITH A LOAD REDUCTION ON 01/02/80 FOR MAINTENANCE ON THE 3E1 HEATER DRAIN PUMP. ON 01/24/80, THE NO. 3 "B" FEEDWATER PUMP TURBINE TRIPPED DURING A ROUTINE OIL PUMP TRIP TEST.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

OYSTER CREEK

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	New Jersey/Ocean 9 miles S of Toms River
2.	Electric Reliability Council	MAAC
3.	Type of Reactor	BWR
4.	Power Level (MWT)	1930
5.	Design Electrical Rating (MWE Net)	650
6.	Date of Initial Criticality	May 3, 1969
7.	Date of Commercial Operation	December 1969
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Barnegat Bay

Utility & Contractor Information

10.	Licensee	Jersey Central Power & Light Co.
11.	Corporate Address	Madison Avenue At Punch Bowl Road Morristown, New Jersey 07960
12.	Corporate Contact	I. Finfrock, Vice President Power Generation
13.	Architect/Engineer	Burns & Roe
14.	Nuclear Steam Supply System	GE
15.	Constructor	J. A. Jones

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	IE Senior Inspector	E. McCabe
19.	IE Principal Inspector	L. Briggs
20.	Licensing Project Manager	S. J. Nowicki
21.	Docket Number	50-219
22.	License Number/Date of Issue	DPR-16, 4/9/69
23.	Public Document Room	Ocean County Library 15 Hooper Avenue Toms River, New Jersey 08753

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

+ Two Core Spray System II Sparger cracks were identified during routine ISI. Licensee is currently evaluating the problem and its resolution.

FACILITY ITEMS (PLANS AND PROCEDURES):

+ None

MANAGERIAL ITEMS:

None.

PLANT STATUS:

+ The plant was shutdown on January 5, 1980 for a scheduled 10 week refueling outage.

+ LAST IE SITE INSPECTION DATE: 2/1/80

+ INSPECTION REPORT NO: 50-219/80-04

SUMMARY

INSPECTION SUMMARY

+ 50-219/79-21 - Nov 27-30: Routine, onsite, unannounced inspection by one region-based inspector (29 hours) of 7 areas: followup of previous inspection findings; review of plant operations; in office LER review; onsite LER followup; IEB followup; and independent effort. A plant tour was also conducted. Noncompliances: None.

+ 50-219/79-24 - Dec 11-14 and 18: Routine, unannounced inspection by two region-based inspectors (55 hours) of 5 inspection areas: followup on previous inspection findings; independent inspection effort; IE Bulletin followup; review of periodic reports; and, fire protection. Noncompliances: One (Storage of flammable material on the 119 foot elevation of the reactor building).

+ 50-219/80-04 - Jan 16: Management meeting held at NRC request to discuss details related to the establishment of an onsite office for the purpose of placing an NRC resident inspector at the facility. All parties concurred on administrative and operational considerations needed to establish an onsite NRC resident inspector office.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

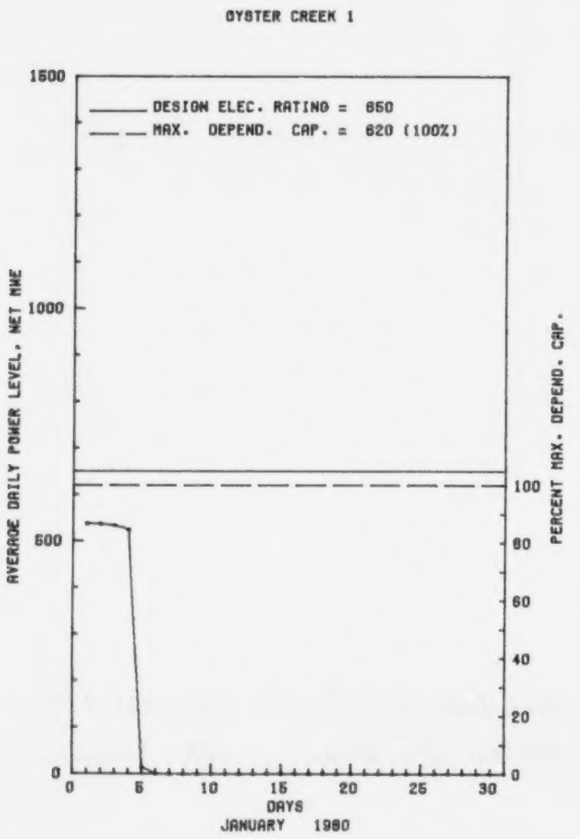
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-008E	12/28/79	01/07/80	Environmental Deviation	One dilution pump off (tripped) when 2 pumps were required for a 20 minute period
79-044/03L	12/19/79	01/18/80	30 Day LER	Reactor building to torus vacuum breaker blocked from opening more than 50 percent due to contractor scaffolding
80-001E	01/05/80	01/15/80	Environmental Deviation	Fish kill during plant shutdown for refueling and maintenance on January 5, 1980
80-001/01T	01/05/80	01/21/80	24 Hours	Failure of 1 of 5 ADS valves to operate during functional testing. Plant was proceeding to cold shutdown at time of event

REVIEWED BY OMPA LOEB: DATE: 3-19-80

REVIEWED BY NRR DOR: S. J. Nowicki DATE: 3-20-80

OYSTER CREEK

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

ITEM 21 CALCULATED
BY USING A WEIGHTED
AVERAGE

1. Docket Number: 50-219
2. Utility Contact: C. M. McClain (201) 455-8748
3. Licensed Thermal Power (MWt): 1930
4. Nameplate Rating (Gross MWe): 687.5 x 0.8 = 550
5. Design Electrical Rating (Net MWe): 650
6. Maximum Dependable Capacity (Gross MWe): 650
7. Maximum Dependable Capacity (Net MWe): 620
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	744.0	88,603.0
12. Hours Reactor Was Critical	101.5	101.5	68,140.8
13. Reactor Reserve Shutdown Hours	.0	.0	465.2
14. Hours Generator On-Line	98.4	98.4	67,493.9
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	157,611	157,611	116,036,769
17. Gross Electrical Energy (MWH)	53,670	53,670	38,335,985
18. Net Electrical Energy (MWH)	48,612	48,612	37,529,625
19. Unit Service Factor	13.2	13.2	76.1
20. Unit Availability Factor	13.2	13.2	76.1
*21. Unit Capacity Factor (MDC Net)	10.5	10.5	70.0
22. Unit Capacity Factor (DER Net)	10.1	10.1	65.2
23. Unit Forced Outage Rate	.0	.0	6.4

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 03/15/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
1	01/05/80	S	645.6	C	1				ANNUAL REFUELING AND MAINTENANCE OUTAGE.

SUMMARY

THE UNIT SHUTDOWN ON 01/05/80 FOR REFUELING.

LEGEND

[1] Type:
F Forced
S Scheduled

[2] Reason:
A–Equipment Failure
B–Maintenance or Test
C–Refueling
D–Regulatory Restriction
E–Operator Training & License Examination
F–Administrative
G–Operational Error
H–Other

[3] Method:
1–Manual
2–Manual Scram
3–Automatic Scram
4–Continuations
5–Load Reduction
9–Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

PALISADES

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Michigan/Van Buren 5 miles S of South Haven
2.	Electric Reliability Council	ECAR
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2530
5.	Design Electrical Rating (MWE Net)	805
6.	Date of Initial Criticality	May 24, 1971
7.	Date of Commercial Operation	December 31, 1971
8.	Condenser Cooling Method	Forced Draft Cooling Towers
9.	Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10.	Licensee	Consumers Power Company
11.	Corporate Address	212 W. Michigan Avenue Jackson, Michigan 49201
12.	Corporate Contact	R. B. DeWitt
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Combustion Engineering
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Senior Inspector	D.C. Boyd
19.	IE Resident Inspector	B. Jorgensen
20.	Licensing Project Manager	R. Silver
21.	Docket Number	50-255
22.	License Number/Date of Issue	DPR-20, 3/24/71
23.	Public Document Room	Kalamazoo Public Library 315 South Rose Street Reference Department Kalamazoo, Michigan 49006

INSPECTION STATUS

ENFORCEMENT STATUS

10 CFR 50.55A(G) REQUIRES INSERVICE TESTING OF ASME CODE CLASS 1, 2 AND 3 VALVES BE PERFORMED IN ACCORDANCE WITH SECTION XI OF THE ASME BOILER AND PRESSURE VESSEL CODE 1974 EDITION AND ADDENDA THROUGH SUMMER 1975. ARTICLE IWV-3410 OF THIS CODE REQUIRES THAT THE VALVE BE TESTED MONTHLY IF AN INCREASE IN STROKE TIME OF 25% OR MORE FROM THE PREVIOUS TEST IS FOUND. CONTRARY TO THE ABOVE, DATA OBTAINED FROM QO-5, "VALVE TEST PROCEDURE" ON JANUARY 17, 1979 AND APRIL 17, 1979, SHOW THAT VALVE CV-0533B, STEAM SUPPLY TO TURBINE DRIVEN AUXILIARY FEEDWATER PUMP, INCREASED FROM 42.2 SECONDS TO 57.0 SECONDS (35% INCREASE) AND THE TEST FREQUENCY WAS NOT INCREASED TO MONTHLY. THIS ITEM APPEARS TO BE REPETITIVE AS IE INSPECTION REPORT 50-255/78-23 CONTAINED A SIMILAR ITEM OF NONCOMPLIANCE REGARDING INSERVICE TESTING OF ASME CODE CLASS 1, 2 AND 3 COMPONENTS.
(7920 INFRACTION)

10 CFR 50, APPENDIX B, CRITERIA XII, CONTROL OF MEASURING AND TEST EQUIPMENT REQUIRES MEASURES BE ESTABLISHED TO ASSURE INSTRUMENTS USED ARE PROPERLY CONTROLLED, CALIBRATED AND ADJUSTED AT SPECIFIED PERIODS TO MAINTAIN ACCURACY. ADMINISTRATIVE PROCEDURE 17.0 REQUIRES TEST EQUIPMENT BE TRACEABLE. QAPP 12-51 REQUIRES TEST EQUIPMENT WHICH IS NOT RECALIBRATED WITHIN ITS SPECIFIED CALIBRATION INTERVAL TO BE CONSIDERED NONCONFORMING AND NOT USED UNTIL CORRECTIVE MEASURES HAVE BEEN TAKEN. CONTRARY TO THE ABOVE, THE SERIAL NUMBER OF THE TEST GAUGE USED IN PERFORMING SURVEILLANCE TEST AO-3 ON JULY 5, 1978, WAS NOT RECORDED. THREE DIFFERENT SERIAL NUMBERS WERE RECORDED FOR THE PERFORMANCE OF AO-3 ON JULY 29, 1979. DEADWEIGHT TESTER SERIAL NUMBER 8428-100018 WAS USED TO TEST A GAUGE ON JUNE 20, 1978, THE CALIBRATION DUE DATE FOR THE TESTER WAS JUNE 13, 1978.
(7920 INFRACTION)

TECHNICAL SPECIFICATION 4.5.3.A, RECIRCULATION HEAT REMOVAL SYSTEMS REQUIRES THE PORTION OF THE SHUTDOWN COOLING SYSTEM THAT IS OUTSIDE THE CONTAINMENT BE TESTED EITHER BY USE IN NORMAL OPERATION OR HYDROSTATICALLY TESTED AT 255 PSIG AT THE INTERVALS SPECIFIED IN 4.5.3.D.(12 MONTHS). CONTRARY TO THE ABOVE PROCEDURE AO-3, SHUTDOWN COOLING SYSTEM TEST, REV. 4, WHICH WAS PERFORMED ON JULY 29, 1979 AND JULY 5, 1978, TO FULFILL THE ABOVE REQUIREMENT TESTED THE PIPING AT LESS THAN 255 PSIG AND DID NOT TEST PIPING OUTSIDE THE CONTAINMENT BETWEEN VALVES 3199 AND 3190 AND MOV3016.
(7920 INFRACTION)

TECHNICAL SPECIFICATION 4.13.C REQUIRES THAT DURING NORMAL PLANT OPERATIONS, PHASE I MEASUREMENTS (REACTOR INTERNALS VIBRATION MONITORING) SHALL BE RECORDED AND ANALYZED ONCE PER SEVEN DAYS. TECHNICAL SPECIFICATION 4.0.2 ALLOWS EXTENSION BY 25% OF THE INTERVAL PROVIDED AND THREE CONSECUTIVE INTERVALS DO NOT EXCEED 3.25 TIMES THE SPECIFIED INTERVAL. CONTRARY TO THE ABOVE, MEASUREMENTS WERE MADE ON JULY 2, 11, 20 AND 31, 1979, WHICH EXCEEDS THE 3.25 TIMES THE INTERVAL FOR THREE INTERVALS AND IN ONE INSTANCE THE 25% ALLOWABLE EXTENSION.
(7920 INFRACTION)

TECHNICAL SPECIFICATION 6.9.2.A(6) REQUIRES THAT A PROMPT REPORT WITH WRITTEN FOLLOWUP WITHIN 14 DAYS BE MADE OF PROCEDURAL INADEQUACIES WHICH PREVENT OR COULD PREVENT, BY ITSELF, THE FULFILLMENT OF THE FUNCTIONS REQUIRED OF SYSTEMS REQUIRED TO COPE WITH AN ACCIDENT ANALYZED IN THE FSAR. CONTRARY TO THE ABOVE PROCEDURE, INADEQUACIES IN THE VALVE CHECK OFF SHEETS OF SURVEILLANCE TEST AO-3, "SHUTDOWN COOLING SYSTEM TESTS", WHICH RESULTED IN VALVES IN THE SODIUM HYDROXIDE PORTIONS OF THE IODINE REMOVAL SYSTEM BEING INADVERTENTLY CLOSED FOR A PERIOD OF TIME ON JULY 29 AND 30, 1979, WAS NOT REPORTED WITHIN THE REQUIRED TIME.
(7920 INFRACTION)

TECHNICAL SPECIFICATION 6.8.1 AND 6.8.3 REQUIRE THAT WRITTEN PROCEDURES FOR SURVEILLANCE ACTIVITIES BE FOLLOWED UNLESS TEMPORARY CHANGES ARE MADE WHICH ARE APPROVED BY TWO MEMBERS OF THE PRC AT LEAST ONE OF WHOM HOLDS A SENIOR REACTOR OPERATORS LICENSE AND THAT THE CHANGE SUBSEQUENTLY BE REVIEWED BY THE PRC AND APPROVED BY THE PLANT SUPERINTENDENT. ADMINISTRATIVE PROCEDURE 10-3-1, "TEMPORARY PROCEDURE CHANGES," WHICH IS ALSO REQUIRED TO BE FOLLOWED BY TECHNICAL SPECIFICATION 6.8.1; ESTABLISHES A SYSTEM TO MAKE THESE CHANGES AND TO INCORPORATE THEM PERMANENTLY INTO THE PROCEDURE WHERE REQUIRED. CONTRARY TO THE ABOVE, ON JULY 5, 1978, CHANGES WERE MADE TO SURVEILLANCE PROCEDURE AO-3, "SHUTDOWN COOLING SYSTEM" TEST WITHOUT REVIEWS, APPROVALS AND DOCUMENTATION REQUIRED ABOVE. THE CHANGE AFFECTED THE AFTER TEST POSITION OF SODIUM HYDROXIDE SYSTEM VALVES 3363ES AND 3364ES AND CORRECTED AN ERROR IN THE PROCEDURE. FAILURE TO MAKE THE CHANGE PROPERLY RESULTED IN THE ERROR STILL EXISTING IN THE PROCEDURE AND IN THESE VALVES BEING INCORRECTLY POSITIONED WHEN THE PROCEDURE WAS PERFORMED A YEAR LATER.
(7920 INFRACTION)

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-44/ 03L-0	07/29/79	01/21/80	30 Day LER	Valves 3363ES and 3364ES were left in closed position, such that the NaOH addition system was not returned to service.
80-01/ 01T-0	01/09/80	01/25/80	14 Day LER	Pipe and pipe support stress per IEB 79-14 do not meet FSAR criteria for either the SSE or OBE.

==

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

- Reactor shutdown on September 10, 1979, to begin a 10-week scheduled refueling and maintenance outage. Start-up tentatively scheduled for late February 1980.

- LAST IE SITE INSPECTION DATE: November 30, 1979

- INSPECTION REPORT NO: 79-24

SUMMARY

INSPECTION SUMMARY

- Inspection on September 17-20; October 17-19, 24, 30; November 7, 30; December 6, 10, 1979; and January 7-9, 1980, (79-14): QA/QC program, implementing procedures, observation of work activities, and review and evaluation of nondestructive examination (NDE) data and reports of inservice inspection (ISI); IE Bulletin 79-13 and IE Bulletin 79-17 activities. The inspection involved 70 inspector hours onsite by two NRC inspectors and 5 hours at Southwest Research Institute. No items of noncompliance or deviations were identified.

- Inspection on October 26 and November 8-9, 15-16 and 21, (79-18): Licensee actions relative to IE Bulletins 79-14 and No. 79-02, including general discussion on NRC requirements, work procedure review, record, review, and observation of work. The inspection involved 24 inspector hours onsite, at the NRC offices, and at the licensee A-E's office by one NRC inspector. No items of noncompliance or deviations were identified.

- Inspection on October 22-26 and 29-30, (79-20): Routine, unannounced inspection of surveillance of safety related components and equipment; implementation of ASME Code Section XI for pumps and valves. The inspection involved 56 inspector hours onsite by one NRC inspector. Of the areas inspected, six items of noncompliance were identified (Control of Test Equipment, 10 CFR 50, Appendix B, Criterion XII; failure to report as required by Technical Specifications 4.5.3.a; failure to test as required by Technical Specifications 4.5.3.a; failure to make a temporary change as required by Technical Specification 6.8.3; failure to conduct surveillance tests at the interval specified by Technical Specification 4.13.c; failure to implement ASME Section XI test requirements for valves).

- Inspection on December 3-7, (79-21)! Routine, unannounced inspection of Environmental Protection, including a review of license management controls; quality control; environmental monitoring program and results and radioactive effluent release rates and limits; Emergency Planning including a review of the site Emergency Plan and implementing procedures; discussions with offsite support agency representatives; review of training, examination of facilities, emergency and first aid kits and supplies; emergency plan drills and critiques; and Confirmatory Measurements including discussion of previous sample results and collection of samples for future comparison. The inspection involved 40 inspector hours onsite by one NRC inspector. Of the 15 areas inspected, no apparent items of noncompliance or deviations were identified in two areas; two apparent items of noncompliance were identified in one area (excessive thermal discharge and discharge below pH limits).

- Inspection during November 1979 (79-22): Routine resident inspection program for November 1979, including procurement, modifications and followup on previously identified items. The inspection involved 100 inspector hours. No items of noncompliance or deviations were identified in any of the inspected areas.

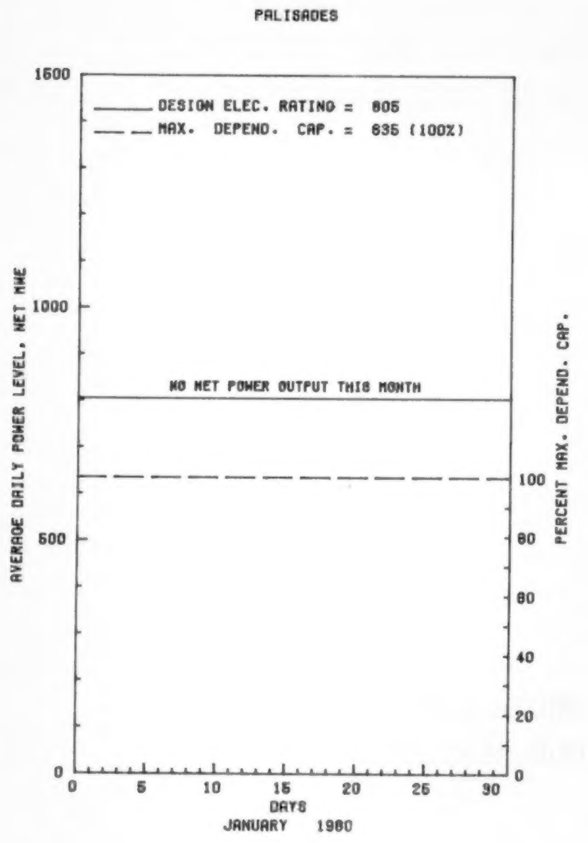
- Inspection on November 30, (79-24): A special management meeting was held to discuss the Consumers Power Company program for the resolution of the problems associated with the breech of containment integrity event of September 11, 1979, and their program for meeting the requirements of the Order of November 9, 1979, modifying the Palisades operating license. The meeting was required by the order. No items of noncompliance were identified as a result of this meeting.

REVIEWED BY OMPA LOEB: MR Beebe **DATE:** 03/19/80

REVIEWED BY NRR DOR: **DATE:** 3/21/80

PALISADES

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-255
2. Utility Contact: P. J. Bodtke (616) 764-8913
3. Licensed Thermal Power (MWt): 2530
4. Nameplate Rating (Gross MWe): 955 x 0.85 = 812
5. Design Electrical Rating (Net MWe): 805
6. Maximum Dependable Capacity (Gross MWe): 675
7. Maximum Dependable Capacity (Net MWe): 635
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	744.0	71,175.0
12. Hours Reactor Was Critical	.0	.0	39,564.8
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	.0	.0	37,426.5
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	0	71,511,480
17. Gross Electrical Energy (MWH)	0	0	22,032,230
18. Net Electrical Energy (MWH)	0	0	20,670,775
19. Unit Service Factor	.0	.0	52.6
20. Unit Availability Factor	.0	.0	52.6
21. Unit Capacity Factor (MDC Net)	.0	.0	45.7
22. Unit Capacity Factor (DER Net)	.0	.0	36.1
23. Unit Forced Outage Rate	.0	.0	36.3

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 03/01/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
13	09/08/79	S	744.0	D	9				TMI MODIFICATIONS AND ANCHOR BOLT INSPECTIONS/REPAIRS ACCORDING TO IE BULLETIN 79-02.

SUMMARY

THE UNIT REMAINED SHUTDOWN THROUGHOUT THE REPORT PERIOD FOR SEISMIC PIPE HANGER/ANCHOR BOLTS INSPECTIONS AND REPAIRS, AND FOR TMI-RELATED MODIFICATIONS.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

PEACH BOTTOM 2

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Pennsylvania/York 19 miles S of Lancaster
2.	Electric Reliability Council	MAAC
3.	Type of Reactor	BWR
4.	Power Level (MWT)	3293
5.	Design Electrical Rating (MWE Net)	1065
6.	Date of Initial Criticality	September 16, 1973
7.	Date of Commercial Operation	July 5, 1974
8.	Condenser Cooling Method	Once Through with Cooling Towers
9.	Condenser Cooling Water Source	Conowingo Pond/Susquehanna River

Utility & Contractor Information

10.	Licensee	Philadelphia Electric Company
11.	Corporate Address	2301 Market Street Philadelphia, Pennsylvania 19101
12.	Corporate Contact	S. L. Deltroff, Vice President Electric Production
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	GE
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	IE Senior Inspector	E. McCabe
19.	IE Resident Inspector	E. Greenman 717-456-7614
20.	Licensing Project Manager	Dick Clark
21.	Docket Number	50-277
22.	License Number/Date of Issue	DPR-44, 8/8/73
23.	Public Document Room	Government Publications Section State Library of Pennsylvania Education Building Commonwealth and Walnut Street Harrisburg, Pennsylvania 17126

INSPECTION STATUS

ENFORCEMENT STATUS

10 CFR 50, APPENDIX B, CRITERION VI STATES IN PART, "MEASURES SHALL BE ESTABLISHED TO CONTROL THE ISSUANCE OF DOCUMENTS, SUCH AS INSTRUCTIONS, PROCEDURES, AND DRAWINGS, INCLUDING CHANGES THERETO...THESE MEASURES SHALL ASSURE THAT DOCUMENTS...ARE DISTRIBUTED TO AND USED AT THE LOCATIONS WHERE THE ACTIVITY IS PERFORMED. THE LICENSEE'S NRC ACCEPTED QUALITY ASSURANCE PLAN, VOLUME III, SECTION 2, PARAGRAPH 6.1 STATES IN PART "...DOCUMENTS, INCLUDING CHANGES...ARE DISTRIBUTED TO AND USED AT THE LOCATION WHERE THE PRESCRIBED ACTIVITY IS PERFORMED." CONTRARY TO THE ABOVE REQUIREMENT CURRENT REVISIONS OF RADWASTE PROCEDURES S.10.4.1.B AND S.10.4.1.C WERE NOT DISTRIBUTED TO AND USED AT THE RADWASTE PANEL, THE LOCATION AT WHICH THE ACTIVITIES DESCRIBED IN THE PROCEDURES WERE PERFORMED.
(7927 INFRACTION)

TECHNICAL SPECIFICATION 6.8.1 REQUIRES THAT WRITTEN PROCEDURES AND ADMINISTRATIVE POLICIES SHALL BE ESTABLISHED, IMPLEMENTED AND MAINTAINED THAT MEET THE REQUIREMENTS OF SECTIONS 5.1 AND 5.3 OF ANSI N18.7-1972 AND APPENDIX "A" OF USAEC REGULATORY GUIDE 1.33. LICENSEE PROCEDURES S.10.4.1.B AND S.10.4.1.C, DEVELOPED PURSUANT TO TECHNICAL SPECIFICATION 6.8.1, EACH REQUIRE THAT A CHECK OFF LIST BE COMPLETED IN THE INITIAL STEP OF THE PROCEDURE. CONTRARY TO THE ABOVE REQUIREMENT THE ACTIVITIES DESCRIBED IN THE PROCEDURES WERE PERFORMED ON SEVERAL OCCASIONS IN OCTOBER 1979 INCLUDING OCTOBER 31, AND NO CHECK OFF LISTS WERE COMPLETED.
(7927 INFRACTION)

CONTRARY TO 10 CFR 55, APP A, 3.D., THE LICENSEE WAS NOT ABLE TO SHOW THAT NINE LICENSED OPERATORS AND SENIOR OPERATORS REVIEWED THE CONTENTS OF ALL ABNORMAL AND EMERGENCY PROCEDURES IN THE PERIOD JANUARY 1977 - NOVEMBER 1979.
(7928 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

- Automatic Depressurization System (ADS) valve nitrogen supply lines were found to have: (1) hard seated check valves on ADS valve accumulator which would leak such that system design capability could not be assured; and (2) supply lines not fully seismically qualified.

FACILITY ITEMS (PLANS AND PROCEDURES):

- None

MANAGERIAL ITEMS:

- An NPR Lessons Learned Team was onsite 1/22-23 to review the licensees implementation of Category "A" Short Term Lessons Learned items.

PLANT STATUS:

- The unit was returned to service 1/9 following a nine day scheduled outage but was shutdown on 1/10 to replace leaking ADS nitrogen supply check valves and perform maintenance on a recirculation pump suction valve. After returning to service 1/19, the reactor operated at a nominal 100 percent power. Representative off-gas and combined Unit 2 and Unit 3 stack gas release rates were 2870 microcuries per second and 942 microcuries per second, respectively.

- LAST IE SITE INSPECTION DATE: 1/31/80
- INSPECTION REPORT NO: 50-277/80-01

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-053/ 01T	12/07/79	12/21/79	24 Hours	Recirculation system sample line isolation valve rendered inoperative by jumper installation
79-054/ 03L	12/13/79	01/11/80	30 Day LER	The 4 Channel B main steam line temperature monitors indicating lower than corresponding channels in the 3 other logic chains
80-001/ 01T	01/03/80	01/17/80	24 Hours	Drywell ventilation valve opened fully (analysis requires maximum 37 degree opening to assure closure during DBA)

SUMMARY

INSPECTION SUMMARY

- 50-277/79-26 - Oct 1-31: Routine, onsite and corporate office regular and backshift inspections by the resident inspector (58 hours). Areas inspected included: refueling activities; physical security; radiation protection for operations/refueling; plant operations; facility tours; control room inspection; LER review onsite; followup on previously identified unresolved items; organization and administration; review of periodic and special reports; and IE Bulletin/Circular followup. Noncompliances: One (Conducting refueling operations without having the mode switch locked in refuel).

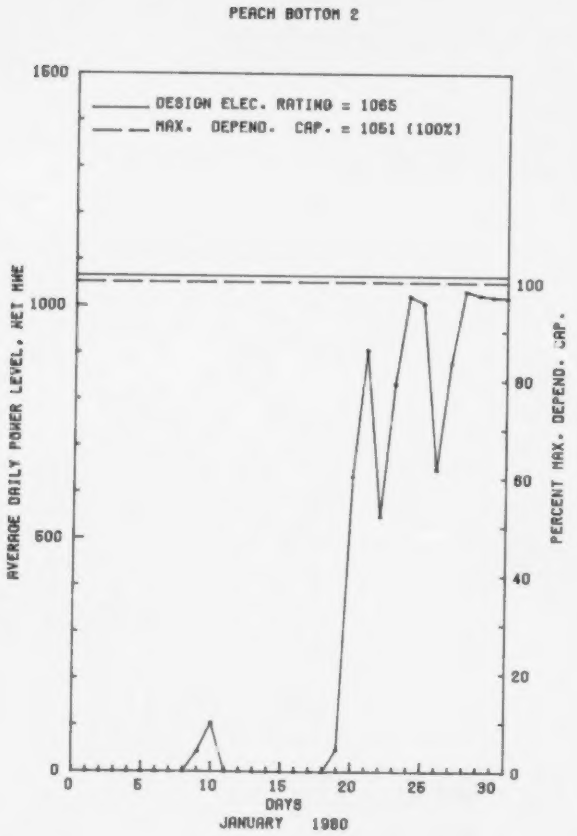
- 50-277/79-29 - Nov 1-30: Routine, onsite and corporate office regular and backshift inspections by the resident inspector (45 hours). Areas inspected included: accessible portions of the Unit 2; radiation protection; maintenance; physical security; plant operations; facility tours; control room inspections; housekeeping; LER review onsite and in-office; followup on previously identified items; review of periodic and special reports, and IE Bulletin followup. Noncompliances: Two (Failure to adhere to Radiation Work Permit (RWP) requirements, and Failure to maintain a fire and penetration seal barrier).

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 3-19-80

REVIEWED BY NRR DOR: Dick Clark **DATE:** 03-21-80

PEACH BOTTOM 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-277
2. Utility Contact: W. M. Alden (215) 841-5022
3. Licensed Thermal Power (MWt): 3293
4. Nameplate Rating (Gross MWe): 1280 x 0.9 = 1152
5. Design Electrical Rating (Net MWe): 1065
6. Maximum Dependable Capacity (Gross MWe): 1098
7. Maximum Dependable Capacity (Net MWe): 1051
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	744.0	48,888.0
12. Hours Reactor Was Critical	347.0	347.0	38,102.3
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	314.0	314.0	37,296.0
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	831,127	831,127	107,596,771
17. Gross Electrical Energy (MWH)	268,680	268,680	35,326,910
18. Net Electrical Energy (MWH)	254,447	254,447	33,870,232
19. Unit Service Factor	42.2	42.2	76.3
20. Unit Availability Factor	42.2	42.2	76.3
21. Unit Capacity Factor (MDC Net)	32.5	32.5	65.9
22. Unit Capacity Factor (DER Net)	32.1	32.1	65.1
23. Unit Forced Outage Rate	3.7	3.7	6.3

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
REFUELING, 3/21/80, ELEVEN WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
13	12/31/79	S	200.7	B	4		CJ	VALVEX	SHUTDOWN CONTINUED FROM 12/31/79 FOR CORE SPRAY FULL-FLOW TEST VALVE.
1	01/09/80	F	12.2	A	3		CH	INSTRU	'A' RFPT CONTROL MALFUNCTION RX LOW LEVEL SCRAM.
2	01/10/80	S	217.1	B	1	2-80-2/1	SF	VALVEX	ADS AIR SUPPLY CHK VALVE LEAKAGE.
3	01/22/80	F	.0	A	5		CB	PUMPXX	RECIRC. PUMP TRIP (LOAD DROP TO 340 MWE).
4	01/25/80	S	.0	H	5		RC	ZZZZZZ	LOAD DROP TO 570 MWE FOR CONTROL ROD ADJUSTMENTS.

SUMMARY

THE UNIT WAS SHUTDOWN UNTIL 01/09/80 TO REPAIR THE CORE SPRAY FLOW TEST VALVE. DURING RETURN TO POWER, A REACTOR SCRAM OCCURRED DUE TO FAILURE OF ELECTRONIC COMPONENTS IN THE FEEDPUMP CONTROL CIRCUIT. THE UNIT WAS TAKEN OFF LINE AGAIN ON 01/10/80 FOR REPLACEMENT OF CHECK VALVES IN THE INSTRUMENT NITROGEN SUPPLY LINE FOR THE AUTOMATIC DEPRESSURIZATION SYSTEM AND OTHER **REPAIRS.** LOAD WAS REDUCED ON 01/22/80 DUE TO LOW LUBE OIL PRESSURE FOR THE RECIRCULATION PUMP MOTOR GENERATOR SET, AND ON 01/25/80 FOR CONTROL ROD ADJUSTMENT. THE UNIT WAS IN POWER COASTDOWN AT THE END OF JANUARY.

LEGEND

[1] Type:
F–Forced
S–Scheduled

[2] Reason:
A–Equipment Failure
B–Maintenance or Test
C–Refueling
D–Regulatory Restriction
E–Operator Training & License Examination
F–Administrative
G–Operational Error
H–Other

[3] Method:
1–Manual
2–Manual Scram
3–Automatic Scram
4–Continuations
5–Load Reduction
9–Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

PEACH BOTTOM 3

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Pennsylvania/York 19 miles S of Lancaster
2.	Electric Reliability Council	MAAC
3.	Type of Reactor	BWR
4.	Power Level (MWT)	3293
5.	Design Electrical Rating (MWE Net)	1065
6.	Date of Initial Criticality	August 7, 1974
7.	Date of Commercial Operation	December 23, 1974
8.	Condenser Cooling Method	Once Through with Cooling Towers
9.	Condenser Cooling Water Source	Conowingo Pond/Susquehanna River

Utility & Contractor Information

10.	Licensee	Philadelphia Electric Company
11.	Corporate Address	2301 Market Street Philadelphia, Pennsylvania 19101
12.	Corporate Contact	S. L. Daltroff, Vice President Electric Production
13.	Architect/Engineer	Bechtel
14	Nuclear Steam Supply System	GE
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	IE Senior Inspector	E. McCabe
19.	IE Resident Inspector	E. Greenman (717-456-7614)
20.	Licensing Project Manager	Dick Clark
21.	Docket Number	50-278
22.	License Number/Date of Issue	DPR-56, 7/2/74
23.	Public Document Room	Government Publications Section State Library of Pennsylvania Education Building Commonwealth and Walnut Street Harrisburg, Pennsylvania 17126

INSPECTION STATUS

ENFORCEMENT STATUS

10 CFR 50, APPENDIX B, CRITERION XVI, STATES IN PART, THAT: "MEASURES SHALL BE ESTABLISHED TO ENSURE THAT...CONDITION ADVERSE TO QUALITY,...ARE PROMPTLY IDENTIFIED AND CORRECTED...." CONTRARY TO THE ABOVE, ON AUGUST 21, 1979, IT WAS IDENTIFIED BY QUALITY CONTROL THAT BOLT PATTERNS FOR THE BASE PLATES OF HANGER NOS. 33HB-H266 AND 33HB-H267 DID NOT CONFORM TO SPECIFIED BOLT PATTERNS ON THE APPLICABLE ISOMETRIC DRAWINGS, HOWEVER, NO CORRECTIVE ACTION WAS INITIATED TO SOLVE AND CORRECT THIS NONCONFORMANCE AS OF THE DATE OF THIS INSPECTION (SEPTEMBER 25, 1979). THIS IS AN INFRACTION.
(7926 INFRACTION)

TS 3.10.A.1 STATES IN PART "THE REACTOR MODE SWITCH SHALL BE LOCKED IN THE "REFUEL" POSITION DURING CORE ALTERATIONS...". CONTRARY TO THE ABOVE, ON 10/9/79, THE UNIT 3 MODE SWITCH WAS FOUND IN REFUELWITH THE KEY IN PLACE & CORE ALTERATIONS HAD OCCURRED PRIOR TO KEY REMOVAL.
(7928 DEFICIENCY)

10 CFR 50, APPENDIX B, CRITERION VI STATES IN PART, "MEASURES SHALL BE ESTABLISHED TO CONTROL THE ISSUANCE OF DOCUMENTS, SUCH AS INSTRUCTIONS, PROCEDURES, AND DRAWINGS, INCLUDING CHANCES THERETO...THESE MEASURES SHALL ASSURE THAT DOCUMENTS...ARE DISTRIBUTED TO AND USED AT THE LOCATION WHERE THE ACTIVITY IS PERFORMED. THE LICENSEE'S NRC ACCEPTED QUALITY ASSURANCE PLAN, VOLUME III, SECTION 2, PARAGRAPH 6.1 STATES IN PART "...DOCUMENTS, INCLUDING CHANGES...ARE DISTRIBUTED TO AND USED AT THE LOCATION WHERE THE PRESCRIBED ACTIVITY IS PERFORMED." CONTRARY TO THE ABOVE REQUIREMENT CURRENT REVISIONS OF RADWASTE PROCEDURES S.10.4.1.B AND S.10.4.1.C WERE NOT DISTRIBUTED TO AND USED AT THE RADWASTE PANEL, THE LOCATION AT WHICH THE ACTIVITIES DESCRIBED IN THE PROCEDURES WERE PERFORMED.
(7930 INFRACTION)

TECHNICAL SPECIFICATION 6.8.1 REQUIRES THAT WRITTEN PROCEDURES AND ADMINISTRATIVE POLICIES SHALL BE ESTABLISHED, IMPLEMENTED AND MAINTAINED THAT MEET THE REQUIREMENTS OF SECTIONS 5.1 AND 5.3 OF ANSI N18.7-1972 AND APPENDIX "A" OF USAEC REGULATORY GUIDE 1.33. LICENSEE PROCEDURES S.10.4.1.B AND S.10.4.1.C, DEVELOPED PURSUANT TO TECHNICAL SPECIFICATION 6.8.1, EACH REQUIRE THAT A CHECK OFF LIST BE COMPLETED IN THE INITIAL STEP OF THE PROCEDURE. CONTRARY TO THE ABOVE REQUIREMENT THE ACTIVITIES DESCRIBED IN THE PROCEDURES WERE PERFORMED ON SEVERAL OCCASIONS IN OCTOBER 1979, INCLUDING OCTOBER 31, AND NO CHECK OFF LISTS WERE COMPLETED.
(7930 INFRACTION)

- CONTRARY TO 10 CFR 55, APP A, 3.D, THE LICENSEE WAS NOT ABLE TO SHOW THAT NINE LICENSED OPERATORS AND SENIOR OPERATORS REVIEWED THE CONTENTS OF ALL ABNORMAL AND EMERGENCY PROCEDURES IN THE PERIOD JANUARY 1977 - NOVEMBER 1979.
(7931 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

+ Automatic Depressurization System (ADS) valve nitrogen supply lines were found to have: (1) hard seated check valves on the ADS valve accumulator which would leak such that system design capability could not be assured; and (2) supply lines not fully seismically qualified.

FACILITY ITEMS (PLANS AND PROCEDURES):

+ None

MANAGERIAL ITEMS:

+ An NRR Lessons Learned Team was onsite 1/22-23 to review the licensees implementation of Category "A" TMI Short Term Lessons Learned items.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-040/ 03L	12/06/79	01/03/80	30 Day LER	"C" core spray disch press SW above TS limit
79-041/ 03L	12/10/79	01/09/80	30 Day LER	While shutdown for maintenance a feedwater flush valve failed to close during surveillance testing making it inoperable

PLANT STATUS:

+ The reactor operated at a nominal 100 percent power during the month. The unit was shutdown for six days starting 1/12 to replace ADS nitrogen supply check valves and repair feedwater heater leaks. The unit was again shutdown on 1/29 to replace a recirculation pump seal. Representative offgas and combined Unit 2 and Unit 3 stack gas release rates were 3960 microcuries per second and 942 microcuries per second, respectively.

+ LAST IE SITE INSPECTION DATE: 1/31/80

+ INSPECTION REPORT NO: 50-278/80-01

SUMMARY

INSPECTION SUMMARY

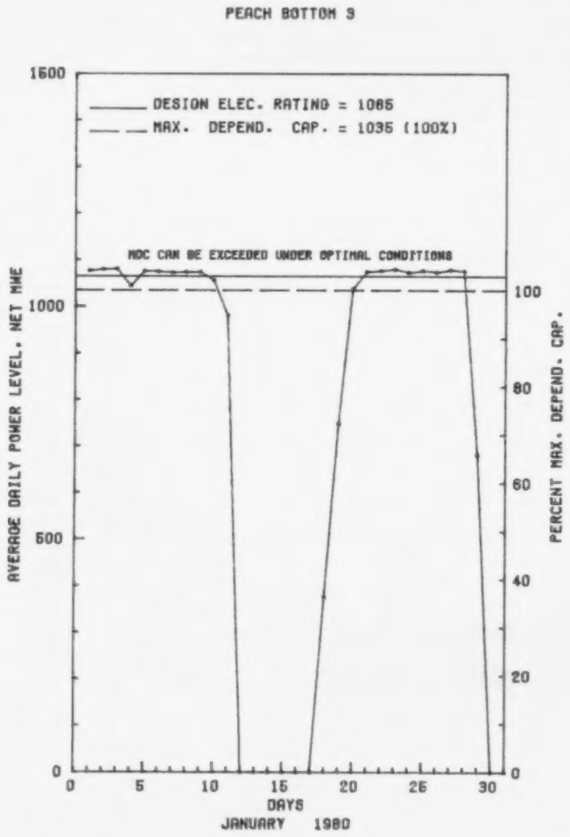
+ 50-278/79-28 - Oct 1-31: Routine, onsite and corporate office regular and backshift inspections by the resident inspector (59 hours). Areas inspected included: refueling; physical security; radiation protection for operations/refueling; plant operations; facility tours; control room inspection; LER review onsite; followup on previously identified unresolved items; organization and administration; review of periodic and special reports; and IE Bulletin/Circular followup. Noncompliances: One (Conducting refueling operations without the mode switch locked in refuel).

+ 50-278/79-32 - Nov 1-30: Routine, onsite and corporate office regular and backshift inspections by the resident inspector (46 hours). Areas inspected included: system restoration from a refueling outage; startup; shutdown margin testing; accessible portions of the Unit; radiation protection; maintenance; physical security; plant operations; facility tours; control room inspections; housekeeping; periodic and special reports; and IE Bulletin followup. Noncompliances: Two (Failure to adhere to Radiation Work Permit (RWP) requirements; and Failure to maintain a fire and penetration seal barrier).

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 3-19-80
REVIEWED BY NRR DOR: Dick Clark **DATE:** 03-21-80

PEACH BOTTOM 3

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket number 50-278
2. Utility Contact: W. M. Alden (215) 841-5022
3. Licensed Thermal Power (MWt): 3293
4. Nameplate Rating (Gross MWe): 1280 x 0.9 = 1152
5. Design Electrical Rating (Net MWe): 1065
6. Maximum Dependable Capacity (Gross MWe): 1098
7. Maximum Dependable Capacity (Net MWe): 1035
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	744.0	44,704.0
12. Hours Reactor Was Critical	565.0	565.0	35,411.0
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	540.7	540.7	34,472.5
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,707,276	1,707,276	95,926,430
17. Gross Elect-ical Energy (MWH)	574,060	574,060	31,159,920
18. Net Electrical Energy (MWH)	553,459	553,459	29,898,259
19. Unit Service Factor	72.7	72.7	77.0
20. Unit Availability Factor	72.7	72.7	77.0
21. Unit Capacity Factor (MDC Net)	71.9	71.9	64.5
22. Unit Capacity Factor (DER Net)	69.8	69.8	62.7
23. Unit Forced Outage Rate	.0	.0	6.7

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 02/06/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
1	01/12/80	S	148.5	B	1		SF	VALVEX	ADS AIR SUPPLY CHK. VALVE LEAKAGE, FEED WATER HTR. LEAK.
2	01/19/80	S	.0	H	5		RC	ZZZZZZ	LOAD DROP TO 640 MWE FOR CONTROL ROD ADJUSTMENTS.
3	01/29/80	S	54.8	B	3		CB	PUMPXX	3B REACTOR RECIRC. PUMP SEAL REPLACEMENT.

SUMMARY

THE UNIT EXPERIENCED LOAD REDUCTIONS ON 01/04/80 DUE TO VIBRATION TRIPPING OF THE 3B REACTOR FEEDPUMP, AND ON 01/10/80 FOR MODIFICATIONS TO THE INSTRUMENT NITROGEN SUPPLY FOR THE AUTOMATIC DEPRESSURIZATION SYSTEM. THE UNIT WAS SHUTDOWN ON 01/12/80 FOR REPLACEMENT OF CHECK VALVES ON THE ADS INSTRUMENT SUPPLY LINE, AND FOR TUBE REPAIRS IN THE 2B FEEDWATER HEATER.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

PILGRIM

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Massachusetts/Plymouth 4 miles SE of Plymouth
2.	Electric Reliability Council	NPCC
3.	Type of Reactor	BWR
4.	Licenses Power Level (MWT)	1998
5.	Design Electrical Rating (MWE Net)	655
6.	Date of Initial Criticality	June 16, 1972
7.	Date of Commercial Operation	December 1972
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Cape Cod Bay

Utility & Contractor Information

10.	Licensee	Boston Edison Company
11.	Corporate Address	800 Boylston Street Boston, Massachusetts 02199
12.	Corporate Contact	A. Corey, Vice President
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	GE
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	IE Senior Inspector	E. McCabe
19.	IE Principal Inspector	W. Baunack
20.	Licensing Project Manager	J. Hannon
21.	Docket Number	50-293
22.	License Number/Date of Issue	DPR-35, 6/8/72
23.	Public Document Room	Plymouth Public Library North Street Plymouth, Massachusetts 02360

INSPECTION STATUS

ENFORCEMENT STATUS

FAILURE TO REPORT EXCEEDING REQUIREMENTS OF TS APP B 1.A. B. FAILURE TO REPORT LOSS OF HPCI SMOKE DETECTOR (T.S. 3.12.A) & LOSS OF SHUTDOWN TRANSFORMER (T.S. 3.9.B.1) AS REQUIRED BY T.S. 6.9.3.2.B. FAILURE TO REPORT ORGANIZATION CHANGE (T.S. 6.2.B) AS REQUIRED BY T.S. 6.9.B.2.E. (7920 DEFICIENCY)

CONTRARY TO TS 6.8.D & PLANT PROCEDURE 1.3.4 PARA III.A. PLANT OPERATING PROCEDURES FOR 5 PROTECTION SYSTEMS (1) FIRE WATER SYSTEM (2) DELUGE & SPRINKLER SYSTEMS (3) CARBON DIOXIDE SYSTEM (4) DRY CHEMICAL SYSTEMS & (5) SMOKE DETECTION SYSTEMS DID NOT ADEQUATELY IMPLEMENT OR WERE NOT IN ACCORDANCE WITH TS 3.12 & 4.12. (7921 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

- Cracks detected on the core spray spargers during inspection.

FACILITY ITEMS (PLANS AND PROCEDURES):

None

MANAGERIAL ITEMS:

None.

PLANT STATUS:

- The facility was shutdown on January 5, 1980 for a refueling outage scheduled to be completed in mid-April.

- LAST IE SITE INSPECTION DATE: 2/8/80

- INSPECTION REPORT NO: 50-293/80-06

SUMMARY

INSPECTION SUMMARY

- 50-293/79-19 - Nov 27-29: Special, unannounced inspection by one region-based inspector (19 hours onsite) of licensee response to IE Bulletin No. 79-19, "Packaging of Low Level Radioactive Waste for Transport and Burial" and plant tours. Noncompliances: None.
- 50-293/79-21 - Dec 10-14: Routine, unannounced inspection by two region-based inspectors (56 hours onsite) of administrative controls for facility procedures, Technical Specification conformance, procedure changes, 10 CFR 50.59 compliance and observation of radiation emergency drill. Noncompliances: One (Failure to adequately implement fire protection operating procedures).
- 50-293/79-22 - Dec 13-14: Special, unannounced inspection by one region-based inspector (19 hours onsite) of new fuel movement activities, inadvertent lift of a spent fuel assembly and plant tours. Noncompliances: None.
- 50-293/80-01 - Jan 8-11: Routine, unannounced inspection by three region-based inspectors (83.5 hours onsite) of administrative control of safety-related maintenance, safety-related maintenance activities, technician qualifications, organization and administration, new fuel receipt and inspection, preparations for refueling, jumper log review and facility tours. Noncompliances: None.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

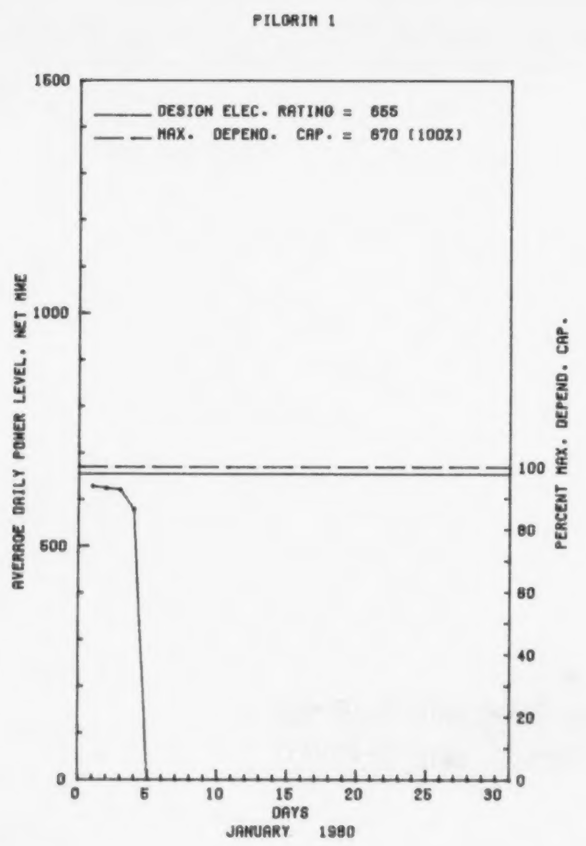
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-047/ 03L	11/17/79	12/17/79	30 Day LER	Fire brigade reduced to less than that required by Technical Specifications
79-048/ 01T	12/24/79	01/04/80	Special	Failed battery chargers for 125 VDC switchyard battery
79-049/ 03L	11/29/79	12/28/79	30 Day LER	Diesel generator declared inop during surveillance testing due to differential relay failure
79-050/ 03L	12/21/79	01/08/80	30 Day LER	HPCI exhaust line snubber discovered to be inoperable

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 3-19-80

REVIEWED BY NRR DOR: John N Hannon **DATE:** 3-21-80

PILGRIM

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-293
2. Utility Contact: G. G. Whitney (617) 746-7900
3. Licensed Thermal Power (MWt): 1998
4. Nameplate Rating (Gross MWe): 780 x 0.87 = 678
5. Design Electrical Rating (Net MWe): 655
6. Maximum Dependable Capacity (Gross MWe): 690
7. Maximum Dependable Capacity (Net MWe): 670
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	744.0	62,640.0
12. Hours Reactor Was Critical	96.1	96.1	45,077.0
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	96.1	96.1	43,648.6
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	181,152	181,152	73,272,264
17. Gross Electrical Energy (MWH)	61,340	61,340	24,226,174
18. Net Electrical Energy (MWH)	58,860	58,860	23,264,983
19. Unit Service Factor	12.9	12.9	69.7
20. Unit Availability Factor	12.9	12.9	69.7
21. Unit Capacity Factor (MDC Net)	11.8	11.8	55.4
22. Unit Capacity Factor (DER Net)	12.1	12.1	56.7
23. Unit Forced Outage Rate	.0	.0	10.7

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 04/07/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
1	01/05/80	S	647.9	C	2		RC	FUELXX	REFUELING OUTAGE.

SUMMARY

THE UNIT WAS SHUTDOWN ON 01/05/80 FOR REFUELING.

LEGEND

[1] Type:
F–Forced
S–Scheduled

[2] Reason:
A–Equipment Failure
B–Maintenance or Test
C–Refueling
D–Regulatory Restriction
E–Operator Training & License Examination
F–Administrative
G–Operational Error
H–Other

[3] Method:
1–Manual
2–Manual Scram
3–Automatic Scram
4–Continuations
5–Load Reduction
9–Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

POINT BEACH 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Wisconsin/Manitowoc 15 miles N of Manitowoc
2.	Electric Reliability Council	
3.	Type of Reactor	PWR
4.	Power Level (MWT)	1518
5.	Design Electrical Rating (MWE Net)	497
6.	Date of Initial Criticality	November 2, 1970
7.	Date of Commercial Operation	December 21, 1970
8.	Condenser Cooling Method	Straight Through
9.	Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10.	Licensee	Wisconsin Electric Power Co.
11.	Corporate Address	231 West Michigan Milwaukee, Wisconsin
12.	Corporate Contact	S. Burstein
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Senior Inspector	R. F. Warnick
19.	IE Resident Inspector	B.L. Jorgnesen
20.	Licensing Project Manager	C. M. Trammell
21.	Docket Number	50-266
22.	License Number/Date of Issue	DPR-24 issued 10/05/70
23.	Public Document Room	Document Department, Library University of Wisconsin-Stevens Point Stevens Point, Wisconsin 54481 Manitowoc Public Library Manitowoc, Wisconsin (Temporary LPDR for spent fuel pool capacity increase amendment only

INSPECTION STATUS

ENFORCEMENT STATUS

ACCORDING TO TECHNICAL SPECIFICATION 15.4.10.1, THE LICENSEE IS REQUIRED AS PART OF THE RADIOLOGICAL ENVIRONMENTAL MONITORING PROGRAM TO ANALYZE SHORELINE SILT SAMPLE NO. 5 FOR GROSS BETA ACTIVITY AND GAMMA SCANNED SEMIANNUALLY. CONTRARY TO THE ABOVE, THE LICENSEE FAILED TO CONDUCT THE REQUIRED ANALYSES OF SHORELINE SILT SAMPLE NO. 5 FOR THE SECOND HALF OF 1978.
(7913 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

- The Unit operated at approximately 77% power this month. This power level is about the maximum obtainable due to reduced primary temperature and pressure limits.

+LAST IE SITE INSPECTION DATE: November 26-30, 1979

+ INSPECTION REPORT NO: 79-18

SUMMARY

INSPECTION SUMMARY

- Inspection on November 26-30, (79-18): Routine, announced inspection of confirmatory measurements, environmental protection, and emergency planning for both units, including licensee programs for quality control of analytical measurements; submission of spiked air particulate and charcoal adsorber for future comparison; quality control of environmental measurements; implementation of environmental protection program; coordination with offsite support agencies; maintenance of emergency equipment, facilities, and supplies; medical arrangements; training; emergency procedures; conduct of tests and drills. The inspection involved 33 inspector hours onsite by one NRC inspector. Of the 15 areas inspected, no apparent items of noncompliance or deviations were identified in 14 areas; an apparent item of noncompliance (failure to analyze shoreline silt sample No. 5 for the second half of 1978) was identified in one area.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

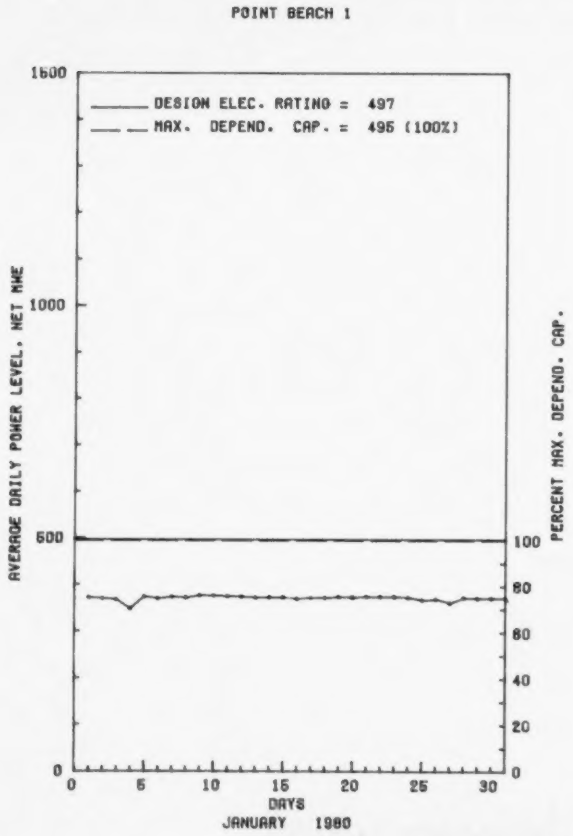
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-21/ 01T-0	12/11/79	12/22/79	14 Day LER	Increasing steam generator primary-to-secondary leakage.
79-22/ 01T-0	12/26/79	01/14/80	14 Day LER	"B" safety injection pump failed to start during operational and system leak test.

REVIEWED BY OMPA LOEB: T.C. Cintula **DATE:** 3-19-80

REVIEWED BY NRR DOR: CM Trammell **DATE:** 3-20-80

POINT BEACH 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

ITEM 21 CALCULATED BY USING A WEIGHTED AVERAGE

1. Docket Number: 50-266
2. Utility Contact: C.W. FAY (414) 277-2811
3. Licensed Thermal Power (MWt): 1518
4. Nameplate Rating (Gross MWe): 582 x 0.9 = 524
5. Design Electrical Rating (Net MWe): 497
6. Maximum Dependable Capacity (Gross MWe): 519
7. Maximum Dependable Capacity (Net MWe): 495
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): 390
10. Reasons for Restrictions, If Any:
CORROSION OF STEAM GENERATOR TUBES

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	744.0	80,976.0
12. Hours Reactor Was Critical	744.0	744.0	67,260.5
13. Reactor Reserve Shutdown Hours	.0	.0	582.2
14. Hours Generator On-Line	744.0	744.0	65,138.0
15. Unit Reserve Shutdown Hours	.0	.0	602.4
16. Gross Thermal Energy (MWH)	885,427	885,427	91,633,222
17. Gross Electrical Energy (MWH)	290,980	290,980	30,924,730
18. Net Electrical Energy (MWH)	276,091	276,091	29,466,262
19. Unit Service Factor	100.0	100.0	80.4
20. Unit Availability Factor	100.0	100.0	81.2
*21. Unit Capacity Factor (MDC Net)	75.0	75.0	74.8
22. Unit Capacity Factor (DER Net)	74.7	74.7	73.2
23. Unit Forced Outage Rate	.0	.0	3.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
STM GEN LEAK TEST & EDDY INSPEC - 2/29/80. (10-15 DAYS)
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	NONE								

SUMMARY

OPERATION WAS ROUTINE AT A 390 MWE (NET) POWER RESTRICTION. THIS WAS DUE TO A SELF-IMPOSED REDUCTION IN CORE AVERAGE TEMPERATURE IN AN EFFORT TO INHIBIT STEAM GENERATOR TUBE CORROSION WITHIN THE TUBESHEET CREVICE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

POINT BEACH 2

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Wisconsin/Manitowoc 15 miles N of Manitowoc
2.	Electric Reliability Council	
3.	Type of Reactor	PWR
4.	Power Level (MWT)	1518
5.	Design Electrical Rating (MWE Net)	497
6.	Date of Initial Criticality	May 30, 1972
7.	Date of Commercial Operation	October 1, 1972
8.	Condenser Cooling Method	Straight Through
9.	Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10.	Licensee	Wisconsin Electric Power Co.
11.	Corporate Address	231 West Michigan Milwaukee, Wisconsin
12.	Corporate Contact	S. Burstein
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Senior Inspector	R. F. Warnick
19.	IE Resident Inspector	B.L. Jorgnesen
20.	Licensing Project Manager	C. M. Trammell
21.	Docket Number	50 301
22.	License Number/Date of Issue	DPR-27 issued 11/16/71
23.	Public Document Room	Document Department, Library University of Wisconsin-Stevens Point Stevens Point, Wisconsin 54481 Manitowoc Public Library Manitowoc, Wisconsin (Temporary LPDR for spent fuel pool capacity increase amendment only

INSPECTION STATUS

ENFORCEMENT STATUS

ACCORDING TO TECHNICAL SPECIFICATION 15.4.10.1, THE LICENSEE IS REQUIRED AS PART OF THE RADIOLOGICAL ENVIRONMENTAL MONITORING PROGRAM TO ANALYZE SHORELINE SILT SAMPLE NO. 5 FOR GROSS BETA ACTIVITY AND GAMMA SCANNED SEMIANNUALLY. CONTRARY TO THE ABOVE, THE LICENSEE FAILED TO CONDUCT THE REQUIRED ANALYSES OF SHORELINE SILT SAMPLE NO. 5 FOR THE SECOND HALF OF 1978.
(7919 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

The plant operated routinely at power during the month.

*LAST IE SITE INSPECTION DATE: November 27-30, 1979

* INSPECTION REPORT NO: 79-20

SUMMARY

INSPECTION SUMMARY

* Inspection on November 26-30, (79-18): Routine, announced inspection of confirmatory measurements, environmental protection, and emergency planning for both units, including licensee programs for quality control of analytical measurements; submission of spiked air particulate and charcoal adsorber for future comparison; quality control of environmental measurements; implementation of environmental protection program; coordination with offsite support agencies; maintenance of emergency equipment, facilities, and supplies; medical arrangements; training; emergency procedures; conduct of tests and drills. The inspection involved 33 inspector hours onsite by one NRC inspector. Of the 15 areas inspected, no apparent items of noncompliance or deviations were identified in 14 areas; an apparent item of noncompliance (failure to analyze shoreline silt sample No. 5 for the second half of 1978) was identified in one area.

* Inspection on November 27-30, (79-20): Routine, unannounced inspection of core power distribution limits; core thermal power evaluation; power coefficient of reactivity measurement; determination of reactivity anomalies; incore/excore calibration; rod drop time test. The inspection involved 20 inspector hours onsite by one NRC inspector. Of the areas inspected, no items of noncompliance or deviations were identified.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

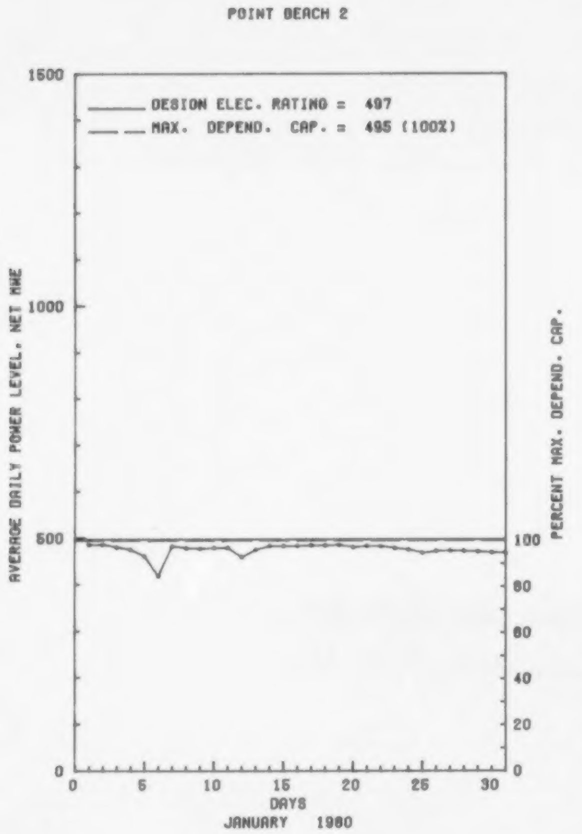
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-08/ 03L-0	12/10/79	12/27/79	30 Day LER	Charging pump taken out of service due to leakage in discharge manifold area.

REVIEWED BY OMPA LOEB: T. C. Cintula DATE: 3-19-80

REVIEWED BY NRR DOR: CM Tramell DATE: 3-20-80

POINT BEACH 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-301
2. Utility Contact: C.W. FAY (414) 277-2811
3. Licensed Thermal Power (MWt): 1518
4. Nameplate Rating (Gross MWe): 582 x 0.9 = 524
5. Design Electrical Rating (Net MWe): 497
6. Maximum Dependable Capacity (Gross MWe): 519
7. Maximum Dependable Capacity (Net MWe): 495
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	744.0	65,761.0
12. Hours Reactor Was Critical	744.0	744.0	59,640.9
13. Reactor Reserve Shutdown Hours	.0	.0	166.1
14. Hours Generator On-Line	744.0	744.0	58,471.0
15. Unit Reserve Shutdown Hours	.0	.0	106.4
16. Gross Thermal Energy (MWH)	1,111,502	1,111,502	78,764,038
17. Gross Electrical Energy (MWH)	369,700	369,700	26,731,040
18. Net Electrical Energy (MWH)	353,404	353,404	25,428,531
19. Unit Service Factor	100.0	100.0	88.9
20. Unit Availability Factor	100.0	100.0	89.1
*21. Unit Capacity Factor (MDC Net)	96.0	96.0	78.9
22. Unit Capacity Factor (DER Net)	95.6	95.6	77.8
23. Unit Forced Outage Rate	.0	.0	1.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUELING - 03/28/80 - 5 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

ITEM 21 CALCULATED BY USING A WEIGHTED AVERAGE

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	NONE								

SUMMARY

OPERATION WAS AT NEAR FULL POWER DURING THE REPORT PERIOD, WITH NO SIGNIFICANT REDUCTIONS OR OUTAGES REPORTED.

LEGEND

[1] Type:
F-Forced
S-Scheduled

[2] Reason:
A-Equipment Failure
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error
H-Other

[3] Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Continuations
5-Load Reduction
9-Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

PRAIRIE ISLAND 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Minnesota/Goodhue 28 miles SE of Minneapolis
2.	Electric Reliability Council	MARCA
3.	Type of Reactor	PWR
4.	Power Level (MWT)	1650
5.	Design Electrical Rating (MWE Net)	530
6.	Date of Initial Criticality	December 1, 1973
7.	Date of Commercial Operation	December 16, 1973
8.	Condenser Cooling Method	Cooling Towers
9.	Condenser Cooling Water Source	Mississippi River

Utility & Contractor Information

10.	Licensee	Northern States Power Company
11.	Corporate Address	414 Nicollet Mall - 8th Floor Minneapolis, Minnesota 55401
12.	Corporate Contact	G. H. Neils (Oper.) L. O. Mayer (Lic.)
13.	Architect/Engineer	Pioneer Services & Eng. Co.
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Northern States Power Co.

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Senior Inspector	R. F. Warnick
19.	IE Resident Inspector	C. D. Feierabend 612-388-8209
20.	Licensing Project Manager	M. Grotenhuis
21.	Docket Number	50-282
22.	License Number/Date of Issue	DPR-42 issued 8/9/73
23.	Public Document Room	Environmental Conservation Library Minneapolis Public Library 300 Nicollet Mall Minneapolis, Minnesota 55401

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

- Operating routinely.

◆LAST IE SITE INSPECTION DATE: November 26-30, 1979

◆ INSPECTION REPORT NO: 79-31

SUMMARY

INSPECTION SUMMARY

- Inspection on November 1-30, (79-29): Routine resident inspection of plant operations, security, radioactive waste system, IE bulletins and circulars and observation of a fire drill. The inspection involved 46 inspector hours by the resident inspector. No items of noncompliance or deviations were identified.

- Inspection on November 26-30, (79-31): Included a review of Security Plan; Security Program Audit; Security Organization; Protected Area Physical Barriers; Vital Area Physical Barriers; Lighting; Access Control (Identification, Authorization and Badging); Access Control (Search); Access Control (Escorting); Access Control (Vital Areas); Alarm Stations; Detection Aids; Assessment Aids; Communications; Testing and Maintenance; Compensatory Measures; Power Supply; Response; Locks, Keys, and Combinations; Records and Reports; and Followup on IE Bulletin No. 7916. The inspection involved 81 inspector hours onsite by two NRC inspectors. The inspection began during the day shift period; 12 inspector hours were accomplished during offshift periods. The licensee was found to be in compliance with NRC requirements within the 20 areas examined during this inspection except for the following six items: Locks, Keys and Combinations - Keys were not properly controlled; Alarm Stations - A required alarm station was not properly manned at all times. Corrected, no response necessary; Access Control, Authorization - Some individuals were not properly authorized access. Closed, no response necessary; Access Control, Search - A visitor was not properly searched; Access Control, Search - Some vehicles were not properly searched. Closed, no response necessary; Protected Area Barriers - The isolation zone was not properly maintained. Closed, no response necessary.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

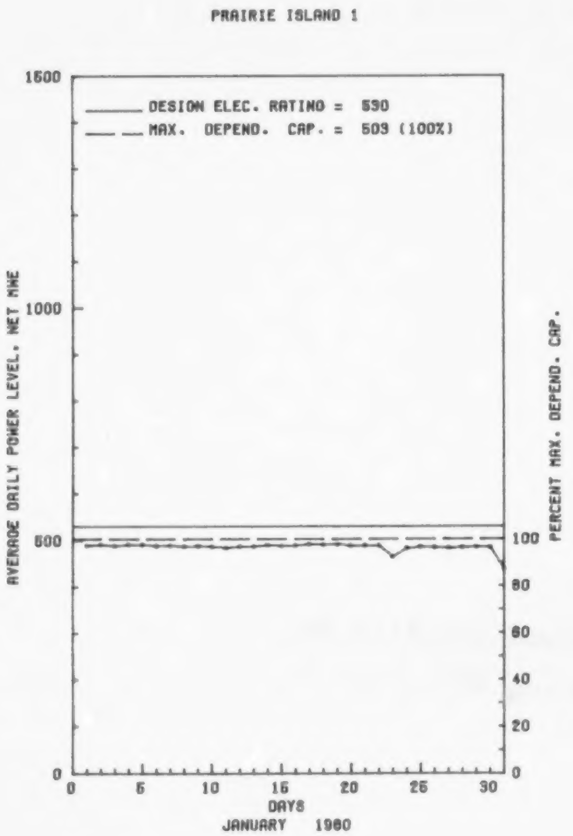
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-30/03L-0	11/21/79	12/21/79	30 Day LER	Two snubbers in the Cooling Water System were found incorrectly installed and one hanger in the feedwater system was damaged.
79-31/03L-0	11/26/79	12/26/79	30 Day LER	Boric Acid Tank Level Test was not performed within the tolerance band.
79-32/03L-0	12/21/79	01/18/80	30 Day LER	During surveillance test, D2 Diesel Generator tripped on high crankcase pressure.
79-33/03L-0	12/28/79	01/14/80	30 Day LER	Auxiliary feedwater pump tripped on overload while closing at the end of the test.

REVIEWED BY CMPA LOEB: T. C. Cintula DATE: 3·19·80

REVIEWED BY NRR DOR: M. Grotenhuis DATE: 3-20-80

PRAIRIE ISLAND 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-282
2. Utility Contact: A. A. Hunstad (612) 388-4767
3. Licensed Thermal Power (MWt): 1650
4. Nameplate Rating (Gross MWe): 659 x 0.9 = 593
5. Design Electrical Rating (Net MWe): 530
6. Maximum Dependable Capacity (Gross MWe): 534
7. Maximum Dependable Capacity (Net MWe): 503
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	744.0	53,712.0
12. Hours Reactor Was Critical	744.0	744.0	41,952.4
13. Reactor Reserve Shutdown Hours	.0	.0	5,531.9
14. Hours Generator On-Line	744.0	744.0	40,867.6
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,214,492	1,214,492	63,293,486
17. Gross Electrical Energy (MWH)	384,920	384,920	20,538,590
18. Net Electrical Energy (MWH)	361,248	361,248	19,200,580
19. Unit Service Factor	100.0	100.0	76.1
20. Unit Availability Factor	100.0	100.0	76.1
21. Unit Capacity Factor (MDC Net)	96.5	96.5	71.1
22. Unit Capacity Factor (DER Net)	91.6	91.6	67.4
23. Unit Forced Outage Rate	.0	.0	13.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUELING, SUMMER 1980, 5 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	01/23/80	F	.0	A	5	80-6	EB	PIPEXX	D2 DIESEL GENERATOR AND NO. 12 DIESEL COOLING WATER PUMP BECAME INOPERABLE AT THE SAME TIME. ORDERLY SHUTDOWN WAS BEGUN AND WAS TERMINATED AT 68% POWER WHEN THE DIESEL GENERATOR WAS AGAIN MADE OPERABLE.

SUMMARY

OPERATION WAS AT NEAR FULL POWER DURING THE REPORT PERIOD, WITH LOAD REDUCTIONS ON 01/23/80 WHEN A DIESEL GENERATOR AND A DIESEL COOLING WATER PUMP BECAME INOPERABLE, AND ON 01/31/80 FOR TURBINE VALVE TESTING.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

PRAIRIE ISLAND 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Minnesota/Goodhue 28 miles SE of Minneapolis
2.	Electric Reliability Council	MARCA
3.	Type of Reactor	PWR
4.	Power Level (MWT)	1650
5.	Design Electrical Rating (MWE Net)	530
6.	Date of Initial Criticality	December 17, 1974
7.	Date of Commercial Operation	December 21, 1974
8.	Condenser Cooling Method	Cooling Towers
9.	Condenser Cooling Water Source	Mississippi River

Utility & Contractor Information

10.	Licensee	Northern States Power Company
11.	Corporate Address	414 Nicollet Mall - 8th Floor Minneapolis, Minnesota 55401
12.	Corporate Contact	G. H. Neils (Oper.) L. O. Mayer (Lic.)
13.	Architect/Engineer	Pioneer Services & Eng. Co.
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Northern States Power Co.

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Senior Inspector	R. F. Warnick
19.	IE Resident Inspector	C. D. Feierabend 612-388-8209
20.	Licensing Project Manager	M. Grotenhuis
21.	Docket Number	50-306
22.	License Number/Date of Issue	DPR-60 issued 10/29/74
23.	Public Document Room	Environmental Conservation Library Minneapolis Public Library 300 Nicollet Mall Minneapolis, Minnesota 55401

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

- Plant shutdown on January 2, 1980, for a scheduled refueling outage.

♦LAST IE SITE INSPECTION DATE: November 26-30, 1979

♦ INSPECTION REPORT NO: 79-25

SUMMARY

INSPECTION SUMMARY

- Inspection on November 1-30, (79-23): Routine resident inspection of plant operations, security, radioactive waste system, IE bulletins and circulars and observation of a fire drill. The inspection involved 46 inspector hours by the resident inspector. No items of noncompliance or deviations were identified.
- Inspection on November 26-30, (79-25): Included a review of Security Plan; Security Program Audit; Security Organization; Protected Area Physical Barriers; Vital Area Physical Barriers; Lighting; Access Control (Identification, Authorization and Badging); Access Control (Search); Access Control (Escorting); Access Control (Vital Areas); Alarm Stations; Detection Aids; Assessment Aids; Communications; Testing and Maintenance; Compensatory Measures; Power Supply; Response; Locks, Keys, and Combinations; Records and Reports; and Followup on IE Bulletin No. 7916. The inspection involved 81 inspector hours onsite by two NRC inspectors. The inspection began during the day shift period; 12 inspector hours were accomplished during offshift periods. The licensee was found to be in compliance with NRC requirements within the 20 areas examined during this inspection except for the following six items: Locks, Keys and Combinations - Keys were not properly controlled; Alarm Stations - A required alarm station was not properly manned at all times. Corrected, no response necessary; Access Control, Authorization - Some individuals were not properly authorized access. Closed, no response necessary; Access Control, Search - A visitor was not properly searched; Access Control, Search - Some vehicles were not properly searched. Closed, no response necessary; Protected Area Barriers - The isolation zone was not properly maintained. Closed, no response necessary.

REPORTS RECEIVED FROM LICENSEE

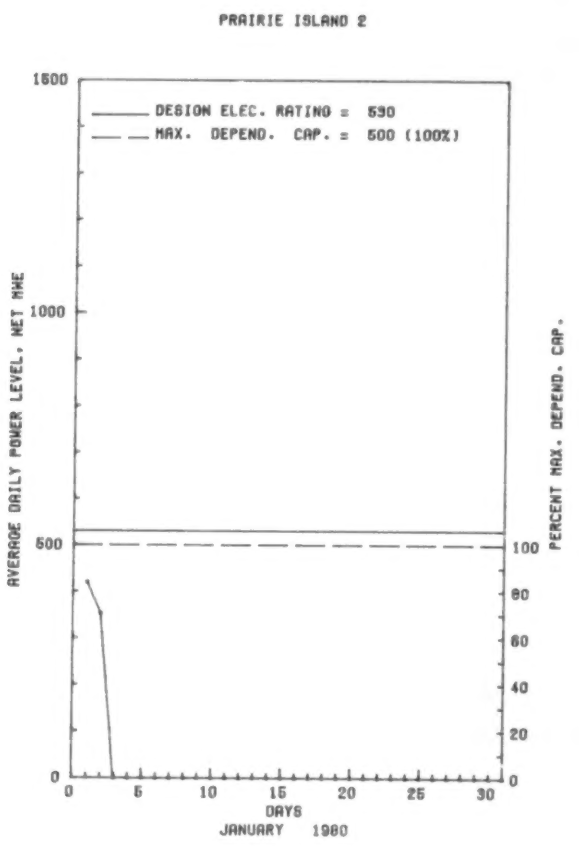
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
80-03/ 01T-0	01/09/80	01/23/80	14 Day LER	In No. 21 steam generator, the Row 1, Column 1 tube in hot leg was found defective.

REVIEWED BY OMPA LOEB: T.C. Cintula DATE: 3·19·80

REVIEWED BY NRR DOR: M Grotenhuis DATE: 3-20-80

PRAIRIE ISLAND 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-306
2. Utility Contact: A. A. Hunstad (612) 388-4767
3. Licensed Thermal Power (MWt): 1650
4. Nameplate Rating (Gross MWe): 659 x 0.9 = 593
5. Design Electrical Rating (Net MWe): 530
6. Maximum Dependable Capacity (Gross MWe): 531
7. Maximum Dependable Capacity (Net MWe): 500
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	744.0	44,830.0
12. Hours Reactor Was Critical	47.9	47.9	39,176.9
13. Reactor Reserve Shutdown Hours	.0	.0	1,406.0
14. Hours Generator On-Line	45.3	45.3	38,446.3
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	60,728	60,728	55,733,231
17. Gross Electrical Energy (MWH)	20,010	20,010	19,074,140
18. Net Electrical Energy (MWH)	16,461	16,461	17,855,093
19. Unit Service Factor	6.1	6.1	85.8
20. Unit Availability Factor	6.1	6.1	85.8
21. Unit Capacity Factor (MDC Net)	4.4	4.4	79.7
22. Unit Capacity Factor (DER Net)	4.2	4.2	75.1
23. Unit Forced Outage Rate	.0	.0	4.9

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 02/17/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	01/02/80	S	698.7	C	2		RC	FUELXX	REFUELING OUTAGE.

SUMMARY

THE UNIT WAS SHUTDOWN ON 01/02/80 FOR REFUELING.

LEGEND

[1] Type:
F Forced
S Scheduled

[2] Reason:
A Equipment Failure
B Maintenance or Test
C Refueling
D Regulatory Restriction
E Operator Training & License Examination
F Administrative
G Operational Error
H Other

[3] Method:
1 Manual
2 Manual Scram
3 Automatic Scram
4 Continuations
5 Load Reduction
9 Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

QUAD CITIES 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Illinois/Rock Island 20 miles NE of Moline
2.	Electric Reliability Council	Main
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2511
5.	Design Electrical Rating (MWE Net)	789
6.	Date of Initial Criticality	October 18, 1971
7.	Date of Commercial Operation	February 18, 1973
8.	Condenser Cooling Method	Spray Canal
9.	Condenser Cooling Water Source	Mississippi River

Utility & Contractor Information

10.	Licensee	Commonwealth Edison Co.
11.	Corporate Address	P. O. Box 767 Chicago, Illinois 60690
12.	Corporate Contact	D. L. Peoples
13.	Architect/Engineer	Sargent and Lundy
14.	Nuclear Steam Supply System	GE
15.	Constructor	United Engineers & Constructors, Inc.

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Senior Inspector	R. Spessard
19.	IE Resident Inspector	N. J. Chrissotimos
20.	Licensing Project Manager	R. B. Bevan
21.	Docket Number	50-254
22.	License Number/Date of Issue	DPR-29 issued 10/1/71
23.	Public Document Room	Moline Public Library 504 17th Street Moline, Illinois 61265

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENTS

+ Installation of a new radwaste solidification system is in progress. Completion is expected in early 1980. (79-26)

PLANT STATUS

Unit operating routinely.

+LAST IE SITE INSPECTION DATE: December 3-4, 1979

+ INSPECTION REPORT NO: 79-29

SUMMARY

INSPECTION SUMMARY

+ Inspection on September 26-27; October 3-4, 22-23 and November 7-9, (79-24): Follow up to IE Bulletin No. 79-02, pipe support base plate designs using concrete expansion anchor bolts; review of test program and procedures; observation of testing anchor bolts; and review of quality records. The inspection involved 68 inspector hours onsite by one NRC inspector. No items of noncompliance were identified in the areas inspected.

+ Inspection on November 5 through December 14, (79-25): Routine, an announced resident inspection of maintenance, plant operations, access control, review and followup on licensee event reports, refueling activities, calibration, radiation protection, independent areas, and an unusual event. The inspection involved 260 inspector hours onsite by one NRC inspector. Of the nine areas inspected, no items of noncompliance were identified.

+ Inspection on November 13-15, (79-26): Nonroutine, announced inspection of licensee actions taken in response to IE Bulletin 79-19 "Packaging of Low-Level Radioactive Waste for Transport and Burial" and, as agreed with the State of South Carolina, verification that identified packaging discrepancies were corrected before resumption of radwaste shipments to Barnwell, South Carolina. The inspection involved 40 inspector hours onsite by two NRC inspectors. No items of noncompliance or deviations were identified.

+ Inspection on November 5-6, (79-27): Included a review of the Security Plan; Security Program Audit; Security Organization; Protected Area Physical Barriers; Vital Area Physical Barriers; Lighting; Access Control (Identification, Authorization and Badging); Access Control (Search); Access Control (Escorting); Access Control (Vital Areas); Alarm Stations; Detection Aids; Assessment Aids; Communications; Testing and Maintenance; Compensatory Measures; Power Supply; Response; Locks, Keys, and Combinations; Records and Reports; Follow-up on previous items of noncompliance; Follow-up on IE Bulletin No. 79-16. The inspection involved 102 inspector hours onsite by three NRC inspectors. The inspection was started during regular working hours. A total of 33 hours were accomplished during off-shift periods. The licensee was found to be in compliance with NRC requirements within the 20 areas examined except for the following items: Physical Barriers/Protected Area - The licensee failed to maintain an adequate isolation zone; and Access Control (Search) - The licensee failed to perform an adequate search of individuals and hand carried items prior to admittance into the protected area.

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-37/ 03L-0	11/16/79	12/11/79	30 Day LER	One of the two Unit 1/2 DG starting air supply valves improperly positioned.
79-39/ 03L-0	12/20/79	01/15/80	30 Day LER	Operation with fuel exposure in excess of the highest fuel exposure for which MAPLHGR limits had been established.
79-40/ 03L-0	12/31/79	01/11/80	30 Day LER	Minimum TS suppression pool level exceeded during operation.
79-41/ 03L-0	12/17/79	01/15/80	30 Day LER	Containment purge isolation valve closure time exceeded TS requirement.
79-42/ 03L-0	12/14/79		30 Day LER	MSIV closure timing exceeded maximum closure time.

==

+ Inspection on December 5-6, (79-28): Licensee actions taken relative to IE Bulletin No. 79-14 including general discussion on NRC requirements, work procedure review and review of records; follow-up on licensee reports of hydraulic snubber seal problems. The inspection involved 8 inspector hours onsite by one NRC inspector. No items of noncompliance or deviations were identified.

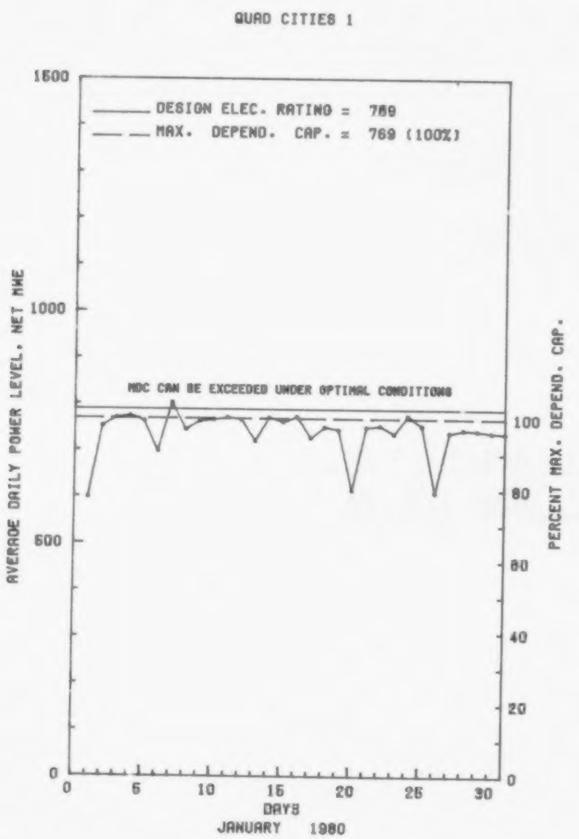
+ Inspection on December 3-4, (79-29): Torus support modification and inservice inspection (ISI); QA/QC program, implementing procedures, work activities, and review and evaluation of data, nondestructive examination (NDE) and welding reports. The inspection involved 8 inspector hours onsite by one NRC inspector. No items of noncompliance or deviations were identified.

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 3-19-80

REVIEWED BY NRR DOR: R B Bevan **DATE:** 3/21/80

QUAD CITIES 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-254
2. Utility Contact: D. Hannum (309) 654-2241
3. Licensed Thermal Power (MWt): 2511
4. Nameplate Rating (Gross MWe): 920 x 0.9 = 828
5. Design Electrical Rating (Net MWe): 789
6. Maximum Dependable Capacity (Gross MWe): 813
7. Maximum Dependable Capacity (Net MWe): 769
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	744.0	67,704.0
12. Hours Reactor Was Critical	744.0	744.0	55,458.1
13. Reactor Reserve Shutdown Hours	.0	.0	3,421.9
14. Hours Generator On-Line	744.0	744.0	52,784.9
15. Unit Reserve Shutdown Hours	.0	.0	909.2
16. Gross Thermal Energy (MWH)	1,747,213	1,747,213	106,291,194
17. Gross Electrical Energy (MWH)	576,126	576,126	34,199,180
18. Net Electrical Energy (MWH)	549,771	549,771	31,965,307
19. Unit Service Factor	100.0	100.0	78.0
20. Unit Availability Factor	100.0	100.0	79.3
21. Unit Capacity Factor (MDC Net)	96.1	96.1	61.4
22. Unit Capacity Factor (DER Net)	93.7	93.7	59.8
23. Unit Forced Outage Rate	.0	.0	8.1

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
28	01/26/80	F	.0	B	5		HG		DEMINX LOAD WAS REDUCED TO CHANGE OVER CONDENSATE DEMINERALIZERS.

SUMMARY

OPERATION WAS ROUTINE DURING THE REPORT PERIOD, WITH A LOAD REDUCTION ON 01/26/80 TO CHANGE OVER CONDENSATE DEMINERALIZER.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

QUAD CITIES 2

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Illinois/Rock Island 20 miles NE of Moline
2.	Electric Reliability Council	Main
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2511
5.	Design Electrical Rating (MWE Net)	789
6.	Date of Initial Criticality	April 26, 1972
7.	Date of Commercial Operation	March 10, 1973
8.	Condenser Cooling Method	Spray Canal
9.	Condenser Cooling Water Source	Mississippi River

Utility & Contractor Information

10.	Licensee	Commonwealth Edison Co.
11.	Corporate Address	P. O. Box 767 Chicago, Illinois 60690
12.	Corporate Contact	D. L. Peoples
13.	Architect/Engineer	Sargent & Lundy
14.	Nuclear Steam Supply System	GE
15.	Constructor	United Engineers & Constructors, Inc.

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Senior Inspector	R. Spessard
19.	IE Resident Inspector	N. J. Chrissotimos
20.	Licensing Project Manager	R. B. Bevan
21.	Docket Number	50-265
22.	License Number/Date of Issue	DPR-30 issued 3/21/72
23.	Public Document Room	Moline Public Library 504 17th Street Moline, Illinois 61265

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENTS

- Installation of a new radwaste solidification system is in progress. Completion is expected in early 1980. (79-23)

PLANT STATUS

The unit was shut down on November 25, 1979 to begin a scheduled 13-week refueling and maintenance outage.

+LAST IE SITE INSPECTION DATE: December 3-4, 1979

+ INSPECTION REPORT NO: 79-26

SUMMARY

INSPECTION SUMMARY

- Inspection on September 26-27; October 3-4, 22-23 and November 7-9, (79-21): Follow up to IE Bulletin No. 79-02, pipe support base plate designs using concrete expansion anchor bolts; review of test program and procedures; observation of testing anchor bolts; and review of quality records. The inspection involved 68 inspector hours onsite by one NRC inspector. No items of noncompliance were identified in the areas inspected.
- Inspection on November 5 through December 14, (79-22): Routine, an announced resident inspection of maintenance, plant operations, access control, review and followup on licensee event reports, refueling activities, calibration, radiation protection, independent areas, and an unusual event. The inspection involved 260 inspector hours onsite by one NRC inspector. Of the nine areas inspected, no items of noncompliance were identified.
- Inspection on November 13-15, (79-23): Nonroutine, announced inspection of licensee actions taken in response to IE Bulletin 79-19 "Packaging of Low-Level Radioactive Waste for Transport and Burial" and, as agreed with the State of South Carolina, verification that identified packaging discrepancies were corrected before resumption of radwaste shipments to Barnwell, South Carolina. The inspection involved 40 inspector hours onsite by two NRC inspectors. No items of noncompliance or deviations were identified.
- Inspection on November 5-6, (79-24): Included a review of the Security Plan; Security Program Audit; Security Organization; Protected Area Physical Barriers; Vital Area Physical Barriers; Lighting; Access Control (Identification, Authorization and Badging); Access Control (Search); Access Control (Escorting); Access Control (Vital Areas); Alarm Stations; Detection Aids; Assessment Aids; Communications; Testing and Maintenance; Compensatory Measures; Power Supply; Response; Locks, Keys, and Combinations; Records and Reports; Follow-up on previous items of noncompliance; Follow-up on IE Bulletin No. 79-16. The inspection involved 102 inspector hours onsite by three NRC inspectors. The inspection was started during regular working hours. A total of 33 hours were accomplished during off-shift periods. The licensee was found to be in compliance with NRC requirements within the 20 areas examined except for the following items: Physical Barriers/Protected Area - The licensee failed to maintain an adequate isolation zone; and Access Control (Search) - The licensee failed to perform an adequate search of individuals and hand carried items prior to admittance into the protected area.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-27/03L-0	11/26/79	12/24/79	30 Day LER	Containment local leak rate tests of several valves revealed leakage rates in excess of TS requirements.
79-28/03L-0	12/05/79	12/31/79	30 Day LER	RCIC area high temp switch trip in excess of TS limits.
79-29/03L-0	12/06/79	01/02/80	30 Day LER	Oxygen analyzer valve failed to fully close.
79-30/03L-0	12/28/79	01/21/80	30 Day LER	RHR service water vault bed plate drain failed to close when float switch was activated.

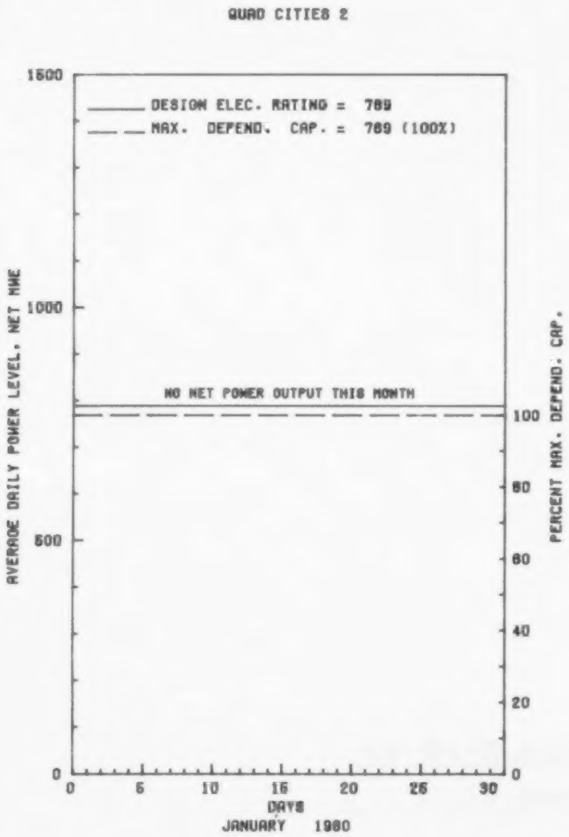
- Inspection on December 5-6, (79-25): Licensee actions taken relative to IE Bulletin No. 79-14 including general discussion on NRC requirements, work procedure review and review of records; follow-up on licensee reports of hydraulic snubber seal problems. The inspection involved 8 inspector hours onsite by one NRC inspector. No items of noncompliance or deviations were identified.
- Inspection on December 3-4, (79-26): Torus support modification and inservice inspection (ISI); QA/QC program, implementing procedures, work activities, and review and evaluation of data, nondestructive examination (NDE) and welding reports. The inspection involved 8 inspector hours onsite by one NRC inspector. No items of noncompliance or deviations were identified.

REVIEWED BY OMPA LOEB: John J Rizzo **DATE:** 3-19-80

REVIEWED BY NRR DOR: RB Bevan **DATE:** 3/21/80

QUAD CITIES 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-265
2. Utility Contact: D. Hannum (309) 654-2241
3. Licensed Thermal Power (MWt): 2511
4. Nameplate Rating (Gross MWe): 920 x 0.9 = 828
5. Design Electrical Rating (Net MWe): 789
6. Maximum Dependable Capacity (Gross MWe): 813
7. Maximum Dependable Capacity (Net MWe): 769
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	744.0	66,819.0
12. Hours Reactor Was Critical	.0	.0	53,128.9
13. Reactor Reserve Shutdown Hours	.0	.0	2,985.8
14. Hours Generator On-Line	.0	.0	50,791.8
15. Unit Reserve Shutdown Hours	.0	.0	702.9
16. Gross Thermal Energy (MWH)	0	0	103,016,109
17. Gross Electrical Energy (MWH)	7	7	32,863,357
18. Net Electrical Energy (MWH)	-13,815	-13,815	30,728,710
19. Unit Service Factor	.0	.0	76.0
20. Unit Availability Factor	.0	.0	77.1
21. Unit Capacity Factor (MDC Net)	.0	.0	59.8
22. Unit Capacity Factor (DER Net)	.0	.0	58.3
23. Unit Forced Outage Rate	.0	.0	0.2

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 03/10/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
13	11/25/79	S	744.0	C	4		RC	FUELXX	UNIT TWO CYCLE 4 REFUELING OUTAGE.

SUMMARY

THE UNIT REMAINED SHUTDOWN THROUGHOUT THE REPORT PERIOD FOR REFUELING.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

RANCHO SECO

FACILITY DATA

Facility Description

1. Location (state/county) (nearest population center)	California/Sacramento 25 miles SE of Sacramento
2. Electric Reliability Council	WSCC
3. Type of Reactor	PWR
4. Power Level (MWT)	2772
5. Design Electrical Rating (MWE Net)	918
6. Date of Initial Criticality	September 16, 1974
7. Date of Commercial Operation	April 17, 1975 (92.6% of full power)
8. Condenser Cooling Method	Closed Cycle
9. Condenser Cooling Water Source	Natural Draft Cooling Towers - Folsom Canal

Utility & Contractor Information

10. Licensee	Sacramento Municipal Utility District
11. Corporate Address	6201 South Street P. O. Box 15830 Sacramento, California
12. Corporate Contact	J. J. Mattimoe Assistant General Manager, Chief Engineer
13. Architect/Engineer	Bechtel
14. Nuclear Steam Supply System	B&W
15. Constructor	Bechtel

Regulatory Information

16. IE Region responsible	V
17. IE Branch Chief	J. L. Crews
18. IE Senior Inspector	B. H. Faulkenberry
19. IE Principal Inspector	H. L. Canter
20. Licensing Project Manager	D. J. Garner
21. Docket Number	50-312
22. License Number/Date of Issue	DPR-54, 8/16/74
23. Public Document Room	Business and Municipal Department Sacramento City-County Library 828 I Street Sacramento, California 95814

R-2

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO THE REQUIREMENTS OF TECHNICAL SPECIFICATION 6.5.2.8.B, THE LICENSEE DID NOT PERFORM AN ANNUAL AUDIT OF THE QUALIFICATIONS OF THE FACILITY STAFF DURING THE YEAR PRECEDING THE DATES OF THE INSPECTION (OCTOBER 29-NOVEMBER 1, 1979).
(7922 DEFICIENCY)

CONTRARY TO THE REQUIREMENTS OF TECHNICAL SPECIFICATION 6.8.3.C, A TEMPORARY CHANGE TO A PROCEDURE SUBJECT TO THIS SPECIFICATION (SFAS DIGITAL CHANNEL 1B REFUELING TEST, SP 203.01B) WAS NOT REVIEWED BY THE PRC AND APPROVED BY THE PLANT SUPERINTENDENT WITHIN SEVEN DAYS OF IMPLEMENTATION (IMPLEMENTED BETWEEN NOV. 18 AND DEC. 18, 1978).
(7922 INFRACTION)

CONTRARY TO THE REQUIREMENTS OF TECHNICAL SPECIFICATION 6.8.3.B, A TEMPORARY CHANGE TO A PROCEDURE SUBJECT TO THIS SPECIFICATION (QUARTERLY MAKEUP PUMP AND VALVES INSPECTION AND SURVEILLANCE TEST, SP 203.02C) WAS MADE AND IMPLEMENTED WITHOUT APPROVAL OF TWO MEMBERS OF THE PLANT MANAGEMENT STAFF (CHANGE APPROVED BY ONE MEMBER OF PLANT MANAGEMENT STAFF (SHIFT SUPERVISOR) AND BY CONTROL ROOM OPERATOR). CHANGE WAS MADE ON MARCH 8, 1978.
(7922 INFRACTION)

TECHNICAL SPECIFICATION TABLE 2.3-1 INDICATES THAT THE NUCLEAR POWER TRIP SETTING IS 5% OF RATED MAXIMUM WHEN THE PLANT IS IN THE SHUT DOWN BYPASS MODE. CONTRARY TO THIS REQUIREMENT, AT 0635 ON NOVEMBER 25, 1979, THE PLANT WAS PLACED IN THE SHUT DOWN BYPASS MODE, BUT THE NUCLEAR POWER TRIP WAS NOT RESET TO 5% OF RATED MAXIMUM.
(7925 INFRACTION)

TECHNICAL SPECIFICATION 6.9.4.1.B REQUIRES A PROMPT REPORT FOR OPERATION WHICH IS LESS CONSERVATIVE THAN THE LEAST CONSERVATIVE LCO. CONTRARY TO THIS REQUIREMENT, A DEGRADED VOLTAGE CONDITION WAS NOTED IN AN OCTOBER 19 LETTER TO THE NRC, BUT IT WAS NOT REPORTED UNTIL NOVEMBER 1, 1979.
(7925 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST (CITE RO REPORT NUMBER, IF APP.)

SYSTEMS AND COMPONENT PROBLEMS:

None.

FACILITY ITEMS (PLANS AND PROCEDURES):

None.

MANAGERIAL ITEMS:

None.

PLANT STATUS:

Plant shutdown on 1/13 for refueling.

LAST IE SITE INSPECTION DATE: 01/02-31/80+

INSPECTION REPORT NO. 50-312/80-03+

NRC POWER RESTRICTIONS: None

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-15-L0	10-31-79	11-16-79	30 Day LER	Additional SPT needed on NSRWS line from DSL Gen HX.
79-20-L0	12-07-79	12-26-79	30 Day LER	Second pinhold leak in socket weld on discharge line from MU pump.
79-21-L0	12-18-79	01-04-80	30 Day LER	APSRS withdrawn seven days early due to conserv error in heat balance.
79-22-T0	12-17-79	01-14-80	14 Day LER	Possible problem closing RX building purge isolation valves.
79-23-L0	12-17-79	01-14-80	30 Day LER	Both cross-tie valves between HPI trains A&B closed when P-238B out/ser.
79-25-L0	12-27-79	01-17-80	30 Day LER	Surveillance of RX building pers hatch cavity done 15 days late.
80-01-L0	01-02-80	01-17-80	30 Day LER	Pipe spt inadequate for suction cross-tie between MU and HPI-B pumps.
80-02-T0	01-04-80	01-16-80	14 Day LER	Pipe spt inadequate making both DHR trains inoperable.

SUMMARY

INSPECTION SUMMARY

- Inspection on July 24-25, August 1-2, August 20, August 27, 1979 and January 10-11, 1980 (Report No. 50-312/79-17)
  Areas Inspected: Routine, unannounced inspection by Regional based inspectors of the licensee's activities performed in response to IE Bulletin 79-02, "Pipe Support Base Plate Designs Using Concrete Expansion Anchor Bolts." The inspection involved 61 inspector hours onsite by two NRC inspectors.

  Results: One item of noncompliance was identified as a result of the inspections.

- Enforcement Conference on November 21, 1979 (Report No. 50-312/79-24)
  An Enforcement Conference was held by representatives of IE Region V and members of SMUD at the Region V Office in Walnut Creek, California. The security inspection and enforcement history at Rancho Seco for the past three years was reviewed with emphasis on recent items of noncompliance.

- Inspection between December 3 & 19, 1979 (Report No. 50-312/79-28)
  Routine inspections of plant operations, physical security, training, followup on previously identified items, and independent inspection effort. The inspection involved 56 inspector hours onsite by the Senior Resident Inspector.

  Results: No items of noncompliance or deviations were identified.

- Inspection on January 17-26, 1980 (Report No. 50-312/80-01)
  Report being prepared; to be reported next month.

- Inspection on January 21, 1980 (Report No. 50-312/80-02)
  Report being prepared; to be reported next month.

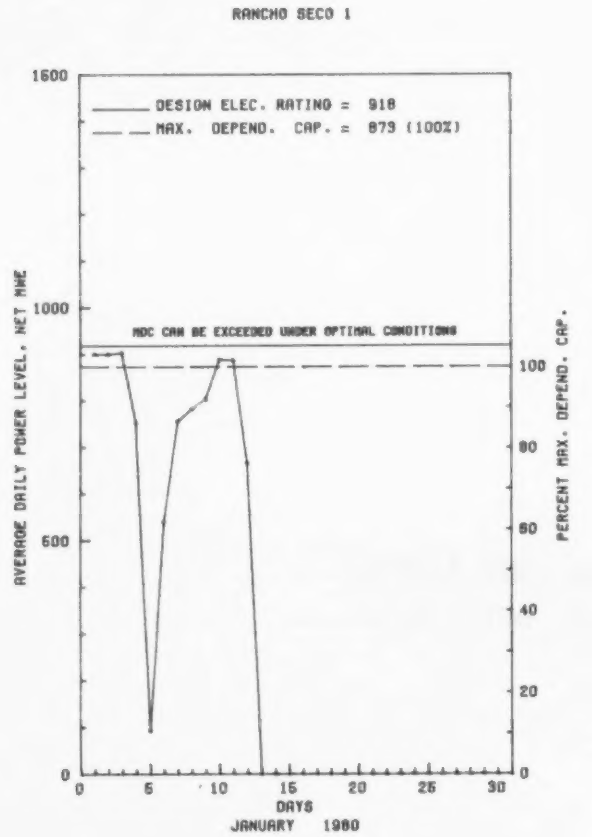
- Inspection on January 2-31, 1980 (Report No. 50-312/80-03)
  Report being prepared; to be reported next month.

REVIEWED BY OMPA LOEB: T.C. Cintula **DATE:** 3·19·80

REVIEWED BY NRR DOR: **DATE:** 3/21/80

RANCHO SECO

AVERAGE DAILY POWEF LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-312
2. Utility Contact: R. W. Colombo (916) 452-3211
3. Licensed Thermal Power (MWt): 2772
4. Nameplate Rating (Gross MWe): 1070 x 0.9 = 963
5. Design Electrical Rating (Net MWe): 918
6. Maximum Dependable Capacity (Gross MWe): 917
7. Maximum Dependable Capacity (Net MWe): 873
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	744.0	42,001.0
12. Hours Reactor Was Critical	271.0	271.0	26,344.5
13. Reactor Reserve Shutdown Hours	.0	.0	3,975.1
14. Hours Generator On-Line	266.9	266.9	25,208.2
15. Unit Reserve Shutdown Hours	.0	.0	1,210.2
16. Gross Thermal Energy (MWH)	689,820	689,820	63,673,838
17. Gross Electrical Energy (MWH)	221,784	221,784	21,506,320
18. Net Electrical Energy (MWH)	206,442	206,442	20,317,332
19. Unit Service Factor	35.9	35.9	60.0
20. Unit Availability Factor	35.9	35.9	62.9
21. Unit Capacity Factor (MDC Net)	31.8	31.8	55.4
22. Unit Capacity Factor (DER Net)	30.2	30.2	52.7
23. Unit Forced Outage Rate	14.5	14.5	31.9

24. Shutdowns Scheduled Over next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
01	01/05/80	F	15.7	D	1	80-002	CF	SUPORT	ANALYSES BY A/E IDENTIFIED SUPPORT WITH SAFETY FACTOR <2.
02	01/12/80	F	29.4	A	2	80-004	CB	VALVEX	RCS LEAK FROM PRESSURIZER SPRAY VALVE HV-21509.
02	01/14/80	S	432.0	C	9		RC	FUELXX	3RD REFUELING OUTAGE COMMENCED AT MIDNIGHT OF JAN. 13, 1980.

SUMMARY

THE UNIT WAS SHUTDOWN ON 01/13/80 FOR REFUELING. ON 01/04/80, LOAD WAS REDUCED FOR PIPE SUPPORT WORK ON THE "B" HIGH PRESSURE INJECTION AND DECAY HEAT REMOVAL SYSTEMS COMMON HEADER.

LEGEND

[1] Type:
- F–Forced
- S–Scheduled

[2] Reason:
- A–Equipment Failure
- B–Maintenance or Test
- C–Refueling
- D–Regulatory Restriction
- E–Operator Training & License Examination
- F–Administrative
- G–Operational Error
- H–Other

[3] Method:
- 1–Manual
- 2–Manual Scram
- 3–Automatic Scram
- 4–Continuations
- 5–Load Reduction
- 9–Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

ROBINSON 2

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	South Carolina/Darlington 5miles NW of Hartsville
2.	Electric Reliability Council	Southeastern
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2300
5.	Design Electrical Rating (MWE Net)	700
6.	Date of Initial Criticality	September 20, 1970
7.	Date of Commercial Operation	March 7, 1971
8.	Condenser Cooling Method	Recirculation from impounded lake
9.	Condenser Cooling Water Source	Robinson Impoundment

Utility & Contractor Information

10.	Licensee	Carolina Power and Light Company
11.	Corporate Address	411 Fayetteville Street Raleigh, North Carolina
12.	Corporate Contact	J. A. Jones Executive Vice President
13.	Architect/Engineer	Ebasco
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Ebasco

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R.C. Lewis
18.	IE Senior Inspector	P. J. Kellogg
19.	IE Principal Inspector	T. McHenry
20.	Licensing Project Manager	D. Neighbors
21.	Docket Number	50-261
22.	License Number/Date of Issue	DPR-23, 7/31/70 (5 Mwt), Full power authorization issued 9/23/70
23.	Public Document Room	Hartsville Memorial Library Home and Fifth Avenue Hartsville, South Carolina 29550

INSPECTION STATUS

ENFORCEMENT STATUS

AS REQUIRED BY TECHNICAL SPECIFICATION 6.8.2, PRIOR TO IMPLEMENTATION, CHANGES TO SAFETY SIGNIFICANT PROCEDURES SHALL BE REVIEWED BY THE PLANT NUCLEAR SAFETY COMMITTEE AND APPROVED BY THE PLANT MANAGER. CONTRARY TO THE ABOVE, DURING THE CYCLE 7 LOW POWER PHYSICS TESTING, STEP 5.29 OF APPENDIX D (CONTROL ROD REACTIVITY WORTH MEASUREMENT) TO PROCEDURE CPL-R-6.8 WAS CHANGED WITHOUT PROPER REVIEW AND APPROVAL. AN ABBREVIATED METHOD WAS USED TO MEASURE THE REACTIVITY WORTH OF CONTROL BANKS C AND D IN OVERLAP.
(7927 DEFICIENCY)

THE FIRE PROTECTION WATER SUPPLY CONNECTION TO THE AUXILIARY BUILDING FIRE HOSE SYSTEM IS NOT PROVIDED WITH A CONTROL VALVE LOCATED ON THE EXTERIOR OF THE BUILDING AND THE NORTHEAST PORTION OF THE EXTERIOR FIRE PROTECTION WATER MAIN IS NOT PROVIDED WITH A SECTIONAL CONTROL VALVE AS REQUIRED BY NFPA-24 WHICH IS REFERENCED BY SECTION 4.0 OF THE LICENSEE'S FIRE PROTECTION PROGRAM REVIEW AS THE APPLICABLE CODE FOR THIS INSTALLATION.
(7929 DEVIATION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

+ UNIT IS IN NORMAL OPERATION.

+ LAST IE SITE INSPECTION DATE: JANUARY 3, 1980

+ INSPECTION REPORT NO: 50-261/80-01

SUMMARY

INSPECTION SUMMARY

+ INSPECTION DECEMBER 3-6 (79-27): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 29 INSPECTOR-HOURS ON SITE IN THE AREAS OF REVIEW OF POST-REFUELING STARTUP AND POWER ESCALATION TESTING. OF THE AREAS INSPECTED, ONE APPARENT ITEM OF NONCOMPLIANCE WAS FOUND (DEFICIENCY - FAILURE TO PROPERLY REVIEW AND APPROVE A CHANGE TO A SAFETY SIGNIFICANT PROCEDURE).

INSPECTION DECEMBER 10-14 (79-29): THIS SPECIAL, UNANNOUNCED INSPECTION INVOLVED 36 INSPECTOR-HOURS ON SITE IN THE AREA OF FIRE PROTECTION/PREVENTION. OF THE AREA INSPECTED, ONE DEVIATION WAS FOUND (INADEQUATE AND SUBSTANDARD CONTROL AND SECTIONAL VALVES FOR FIRE PROTECTION WATER SYSTEM).

INSPECTION DECEMBER 11-14 (79-30): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 27 INSPECTOR-HOURS ON SITE IN THE AREAS OF PROCEDURES, PLANT OPERATIONS, LICENSEE EVENT FOLLOWUP, AND FOLLOWUP ON IE BULLETINS, CICULARS AND NOTICES. OF THE FOUR AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

- INSPECTION JANUARY 3 (80-01): THIS SPECIAL, ANNOUNCED INSPECTION INVOLVED 7 INSPECTOR-HOURS ON SITE IN THE AREA OF STEAM GENERATOR FEEDWATER LINE RADIOGRAPHY (RT) - BULLETIN 79-13. NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
NONE				

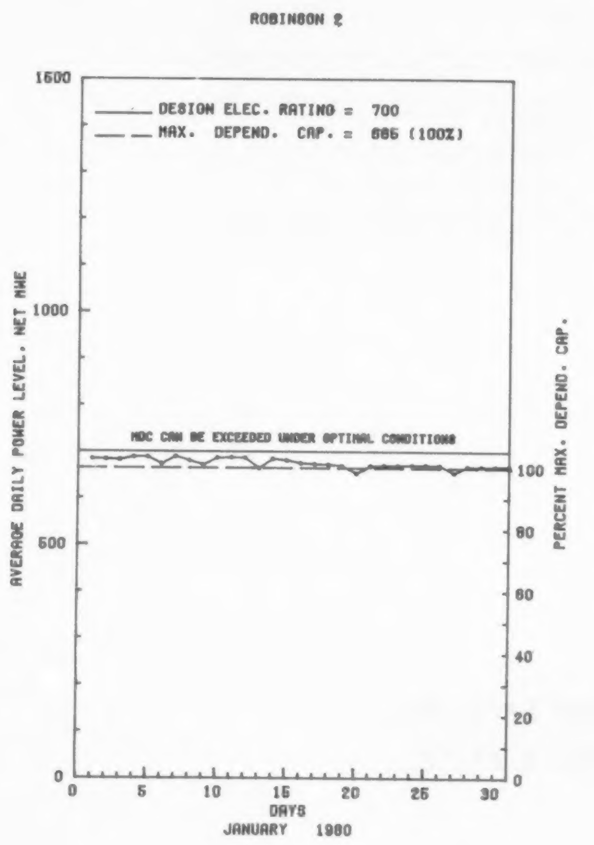
REVIEWED BY OMPA LOEB: T.C. Cintula **DATE:** 3-19-80

REVIEWED BY NRR DOR: Don Neighbors **DATE:** 3-20-80

GPO 871-667

ROBINSON 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-261
2. Utility Contact: M. L. Watford (803) 383-4524
3. Licensed Thermal Power (MWt): 2300
4. Nameplate Rating (Gross MWe): 854 x 0.9 = 769
5. Design Electrical Rating (Net MWe): 700
6. Maximum Dependable Capacity (Gross MWe): 700
7. Maximum Dependable Capacity (Net MWe): 665
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	744.0	78,126.0
12. Hours Reactor Was Critical	744.0	744.0	60,958.9
13. Reactor Reserve Shutdown Hours	.0	.0	719.3
14. Hours Generator On-Line	744.0	744.0	59,434.9
15. Unit Reserve Shutdown Hours	.0	.0	23.2
16. Gross Thermal Energy (MWH)	1,593,955	1,593,955	122,144,284
17. Gross Electrical Energy (MWH)	526,758	526,758	39,463,271
18. Net Electrical Energy (MWH)	502,084	502,084	37,407,186
19. Unit Service Factor	100.0	100.0	76.1
20. Unit Availability Factor	100.0	100.0	76.1
21. Unit Capacity Factor (MDC Net)	101.5	101.5	72.0
22. Unit Capacity Factor (DER Net)	96.4	96.4	68.4
23. Unit Forced Outage Rate	.0	.0	13.0

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUELING/MAINTENANCE, MAY 1980, 6 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	NONE								

SUMMARY

OPERATION WAS AT NEAR FULL POWER THROUGHOUT THE REPORT PERIOD, WITH NO OUTAGES OR REDUCTIONS REPORTED. POWER LEVEL WAS SLIGHTLY RESTRICTED DUE TO EXCESSIVE MOISTURE CARRYOVER TO THE HIGH PRESSURE TURBINE.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

best copy available

SALEM 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	New Jersey/Salem 20 miles S of Wilmington, Del.
2.	Electric Reliability Council	NPCC
3.	Type of Reactor	PWR
4.	Power Level (MWT)	3338
5.	Design Electrical Rating (MWE Net)	1090
6.	Date of Initial Criticality	December 11, 1976
7.	Date of Commercial Operation	June 30, 1977
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Delaware River

Utility & Contractor Information

10.	Licensee	Public Service Electric & Gas Co.
11.	Corporate Address	80 Park Place Newark, New Jersey 07101
12.	Corporate Contact	F. P. Librizzi General Manager - Electric Prod.
13.	Architect/Engineer	PSE&G
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	UE&C

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	IE Senior Inspector	R. R. Keimig
19.	IE Resident Inspector	L. J. Norrholm (609-935-3850) (FTS 341-3000)
20.	Licensing Project Manager	W. Ross
21.	Docket Number	50-272
22.	License Number/Date of Issue	DPR-70, 8/13/76
23.	Public Document Room	Salem Free Public Library 112 West Broadway Salem, New Jersey 08079

INSPECTION STATUS

ENFORCEMENT STATUS

TECHNICAL SPECIFICATION 6.8.1 STATES IN PART: "WRITTEN PROCEDURES SHALL BE ESTABLISHED, IMPLEMENTED AND MAINTAINED COVERING...THE APPLICABLE PROCEDURES RECOMMENDED IN APPENDIX A OF REGULATORY GUIDE 1.33, NOVEMBER 1972." REGULATORY GUIDE 1.33 INCLUDES PARAGRAPH A.3 EQUIPMENT CONTROL (E.G. LOCKING AND TAGGING). THE IMPLEMENTING PROCEDURE, OPERATIONS DIRECTIVES MANUAL REVISION 5, STATES, IN SECTION 2.4.5, "...WHEN A NEW VALVE LINEUP IS PERFORMED, THE OPERATOR IS TO POSITION ALL VALVES AS SPECIFIED BY THE VALVE LINEUP AND INITIAL THE APPROPRIATE SPACE....THE COMPLETED VALVE LINEUP WILL THEN BE USED TO RECORD VALVE DEVIATIONS RESULTING FROM TAGGING OR VALVE MANIPULATIONS FOR SPECIAL PLANT CONDITIONS....WHEN A VALVE LINEUP IS DEVIATED FROM DUE TO TAGGING OR SPECIAL PLANT CONDITIONS, THE ON DUTY CONTROL OPERATOR WILL NOTE IN THE "C&T FOR OR OUT OF POSITION COLUMN" THE REASON...." CONTRARY TO THE ABOVE, ON NOVEMBER 1, 1979, MANUAL VALVES 11CS6 AND 12CS6 WERE FOUND TO BE SHUT. THE VALVE LINEUP AND DEVIATION SHEET INDICATED THAT THE VALVES WERE LOCKED OPEN AS SPECIFIED. THE LINEUP HAD BEEN COMPLETED AND INITIALED ON OCTOBER 27, 1979. NO DEVIATION FROM THE REQUIRED POSITION WAS NOTED.
(7928 INFRACTION)

IN CORRESPONDENCE TO THE DIRECTOR, OFFICE OF INSPECTION AND ENFORCEMENT, REGION I, DATED OCTOBER 11, 1979, RELATING TO IE BULLETIN 79-07 (SEISMIC STRESS ANALYSIS OF SAFETY RELATED PIPING), THE LICENSEE STATES, "...PRIOR TO ENTERING MODES 3 AND 4, THE FOLLOWING WORK WILL BE ACCOMPLISHED:...3) RE-EVALUATION OF THE SUPPORTS, NOZZLES, AND PENETRATIONS FOR ENTIRE AUXILIARY FEEDWATER SYSTEM...5) FIELD MODIFICATIONS TO SUPPORTS AND PENETRATIONS EVALUATED IN (2), (3) & (4) THAT FAIL TO MEET OUR CRITERIA STATED IN OUR SEPTEMBER 21, 1979 SUBMITTAL..." CONTRARY TO THE ABOVE, SALEM UNIT 1 ENTERED MODE 4 AT 0006 ON NOVEMBER 13, 1979 PRIOR TO COMPLETING REQUIRED MODIFICATIONS TO TWO SEISMIC SUPPORTS IN THE AUXILIARY FEEDWATER SYSTEM. THE SUPPORTS, AFS-156 AND AFS-157, ARE LOCATED ON THE SUCTION PIPING TO # 12 AUXILIARY FEEDWATER PUMP, AND WERE IDENTIFIED TO THE STATION AS REQUIRING REPAIR TO MEET DESIGN CRITERIA AT APPROXIMATELY 1545 ON NOVEMBER 13, 1979.
(7928 DEVIATION)

10 CFR 50, APPENDIX B, CRITERIA VI REQUIRES MEASURES BE ESTABLISHED FOR DOCUMENT CONTROL & REVIEW OF DOCUMENTS FOR ADEQUACY. THE LICENSEES MEASURES ESTABLISHED INCLUDE: QA MANUAL INSTRUCTION QAI 6-1, "DOCUMENT CONTROL" & SALEM QA PROGRAM PROCEDURE AP-17 SECTION 6.0 IF AO-17, "DOCUMENT CONTROL" ADDRESSES THE ABOVE. CONTRARY TO THE ABOVE, THE FOLLOWING WAS NOTED DURING A REVIEW OF RAD WASTE DOCUMENT CONTROL IMPLEMENTATION: - AS OF 11/13/79, NO CURRENT COPY OF 49 CFR PART 100 TO 199 WAS ONSITE. - TWO YEAR OLD COPIES OF 10 CFR 20 & 49 CFR 100 TO 199 WERE IN USE AT LOCATION OF ACTIVITY. - NO FORMALIZED METHOD OF DOCUMENT CONTROL WAS IN PLACE TO ASSURE IMPLEMENTATION OF ABOVE.
(7931 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

+ A 2 gph leak in the spent fuel pool liner has been detected. The leak source has not been determined.

FACILITY ITEMS (PLANS AND PROCEDURES):

+ None

MANAGERIAL ITEMS:

None

PLANT STATUS:

+ Operating at full power.

+ LAST IE SITE INSPECTION DATE: 2/8/80

+ INSPECTION REPORT NO: 50-272/80-04

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-058/ 01T	09/07/79	11/10/79	24 Hours	Potential unreviewed safety question associated with environmental qualifications of non-safety related control systems
79-068/ 03L	11/28/79	12/20/79	30 Day LER	During cold shutdown #12 safety injection pump was started and the associated AC breaker tripped investigation revealed a bound shaft
79-069/ 03L	11/21/79	12/20/79	30 Day LER	While conducting a containment purge during Refueling the air particulate detector motor failed rendering the radiation monitor inoperable
79-071/ 03L	10/31/79	11/30/79	30 Day LER	During mode 5 operations, all three diesel generators were tagged out as inoperable due to an operator error
79-072/ 03L	11/05/79	12/03/79	30 Day LER	Meteorological instruments were out of service for 74 hours due to a failed inverter in the power supply
79-075/ 03L	11/23/79	12/20/79	30 Day LER	While performing an air lock leak rate test the overall leak rate was found to be in excess of allowable limits

SUMMARY

INSPECTION SUMMARY

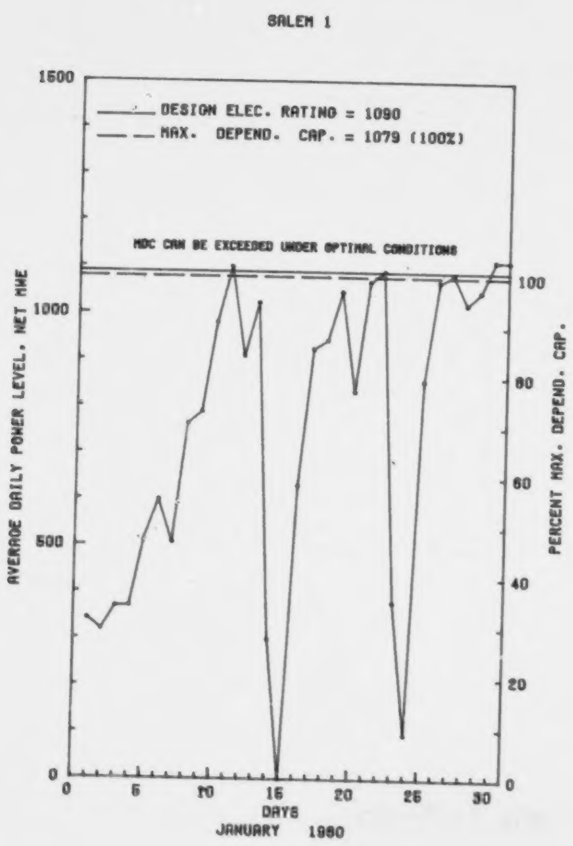
+ 50-272/79-31 - Nov 13-15: Special, unannounced inspection by two region-based inspectors (33 hours) of the actions taken by the licensee in response to IE Bulletin No. 79-19, "Packaging of Low-Level Radioactive Waste for Transport and Burial", including: document control and review; radioactive waste management; personnel organization; audits; radioactive waste solidification; and followup on previous enforcement action. Noncompliances: One (Failure to provide document control and adequate procedures in accordance with 10 CFR 50).

REVIEWED BY OMPA LOEB: T.C. Cintula DATE: 3-19-80

REVIEWED BY NRR DOR: W Ross DATE: 3-20-80

SALEM 1

AVERAGE DAILY POWER LEVEL(MWe)



OPERATING STATUS

1. Docket Number: 50-272
2. Utility Contact: L. K. Miller (609) 365-7000
3. Licensed Thermal Power (MWt): 3338
4. Nameplate Rating (Gross MWe): 1300 x 0.9 = 1170
5. Design Electrical Rating (Net MWe): 1090
6. Maximum Dependable Capacity (Gross MWe): 1124
7. Maximum Dependable Capacity (Net MWe): 1079
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	744.0	22,705.0
12. Hours Reactor Was Critical	696.3	696.3	10,784.6
13. Reactor Reserve Shutdown Hours	.0	.0	22.7
14. Hours Generator On-Line	681.7	681.7	10,212.8
15. Unit Reserve Shutdown Hours	.0	.0	0
16. Gross Thermal Energy (MWH)	1,789,716	1,789,716	29,326,012
17. Gross Electrical Energy (MWH)	586,080	586,080	9,753,580
18. Net Electrical Energy (MWH)	556,282	556,282	9,185,872
19. Unit Service Factor	91.6	91.6	45.0
20. Unit Availability Factor	91.6	91.6	45.0
21. Unit Capacity Factor (MDC Net)	69.3	69.3	37.5
22. Unit Capacity Factor (DER Net)	68.6	68.6	37.1
23. Unit Forced Outage Rate	8.4	8.4	43.4

24. Shutdowns Scheduled Over Next 6 Months (Type,Date,and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-001	01/01/80	F	.0	B	5		SH	ZZZZZZ	NRC TECH. SPECS./REG. GUIDES.
80-002	01/03/80	F	.0	B	5		SH	ZZZZZZ	NRC TECH. SPECS./REG. GUIDES.
80-003	01/05/80	F	.0	B	5		SH	ZZZZZZ	NRC TECH. SPECS./REG. GUIDES.
80-004	01/07/80	F	.0	B	5		SH	ZZZZZZ	NRC TECH. SPECS./REG. GUIDES.
80-005	01/08/80	F	.0	B	5		SH	ZZZZZZ	NRC TECH. SPECS./REG. GUIDES.
80-009	01/11/80	F	.0	A	5		HH	HTEXCH	CLEAN CONDENSATE PUMPS SUCTION STRAINERS.
80-015	01/14/80	F	37.1	A	3		RB	ZZZZZZ	NUCLEAR INSTRUMENTATION (SPURIOUS NOISE SPIDE ON POWER RANGE CHANNEL N-43 WHILE N-44 WAS IN TEST).
80-017	01/17/80	F	.0	A	5		HH	HTEXCH	CLEAN CONDENSATE PUMPS SUCTION STRAINERS.
80-019	01/20/80	F	.0	A	5		HH	HTEXCH	CLEAN CONDENSATE PUMPS SUCTION STRAINERS.
80-022	01/23/80	F	.0	A	5		HH	HTEXCH	CLEAN CONDENSATE PUMPS SUCTION STRAINERS.
80-024	01/23/80	F	25.2	A	3		HH	TRANSF	LOSS OF AUXILIARY TRANSFORMER.
80-026	01/27/80	F	.0	A	5		HH	HTEXCH	CLEAN CONDENSATE PUMPS SUCTION STRAINERS.

SUMMARY

POWER ESCALATION TESTING CONTINUED FROM LAST PERIODS MAINTENANCE OUTAGE UNTIL 01/26/80. OPERATION WAS AT NEAR FULL POWER THE REMAINDER OF THE REPORT PERIOD.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

SAN ONOFRE 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	California/San Diego 5 miles S of San Clemente
2.	Electric Reliability Council	WSCC
3.	Type of Reactor	PWR
4.	Power Level (MWT)	1347
5.	Design Electrical Rating (MWE Net)	436
6.	Date of Initial Criticality	June 14, 1967
7.	Date of Commercial Operation	January 1, 1968
8.	Condenser Cooling Method	Circulating sea water
9.	Condenser Cooling Water Source	Pacific Ocean

Utility & Contractor Information

10.	Licensee	Southern California Edison Co.
11.	Corporate Address	2244 Walnut Grove Avenue Rosemead, California 91770
12.	Corporate Contact	R. E. Woodbury V.P. & General Counsel
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	V
17.	IE Branch Chief	J. L. Crews
18.	IE Senior Inspector	B. H. Faulkenberry
19.	IE Principal Inspector	L. G. Miller
20.	Licensing Project Manager	A. Burger
21.	Docket Number	50-206
22.	License Number/Date of Issue	DPR-13, 3/27/67
23.	Public Document Room	Mission Viejo Branch Library 24851 Chrisanta Drive Mission Viejo, California 92676

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO 10 CFR 50, APPENDIX B, CRITERION XVI, AS AMPLIFIED BY SECTION D.16 OF THE FSAR AND SECTION 12.0 OF THE QA MANUAL, A CORRECTIVE ACTION REQUEST WAS NOT INITIATED TO IDENTIFY THE MISSING FIRE HOSES FOUND DURING THE FIRE HOSE INSPECTION OF APRIL 5, 1978.
(7901 DEFICIENCY)

CONTRARY TO TS 6.4.1 AND SECTION 5.5 OF ANSI N18.1-1971, NO TRAINING PROGRAM FOR THE RETRAINING OF MAINTENANCE PERSONNEL HAD BEEN ESTABLISHED.
(7901 DEFICIENCY)

CONTRARY TO TS 6.8.1 AND APPENDIX A OF REGULATORY GUIDE 1.33, REVISION 1, WRITTEN PROCEDURES FOR A COMPREHENSIVE PREVENTIVE MAINTENANCE PROGRAM WERE NOT PROVIDED.
(7901 DEFICIENCY)

CONTRARY TO TS 6.5.1.7.B, THE ON-SITE REVIEW COMMITTEE WAS NOT PROVIDING THE PLANT MANAGER WITH A WRITTEN DETERMINATION REGARDING THE UNREVIEWED SAFETY QUESTION ISSUE FOR THE PROCEDURES AND PROCEDURE CHANGES THEY REVIEWED.
(7901 DEFICIENCY)

CONTRARY TO TS 6.5.2.8, THE NUCLEAR AUDIT AND REVIEW COMMITTEE FAILED TO AUDIT THE PERFORMANCE, TRAINING AND QUALIFICATIONS OF FACILITY STAFF MEMBERS OTHER THAN THE LICENSED OPERATORS. THE COMMITTEE ALSO FAILED TO AUDIT CONFORMANCE TO TS 3.3.4 IN 1973.
(7901 INFRACTION)

CONTRARY TO TS 6.8.1 AND BTP APCSB 9.5-1, LIVE FIRE FIGHTING DEMONSTRATIONS WERE NOT HELD DURING 1978 AND FIRE BRIGADE MONTHLY TRAINING MEETINGS WERE NOT HELD BETWEEN SEPTEMBER 15 AND NOVEMBER 2, 1978.
(7901 DEFICIENCY)

CONTRARY TO TS 6.8.1 AND SECTION 5.3 OF ANSI N18.7-1976: (1) CERTAIN DIVISION ORDERS AND STATION ORDERS DID NOT CONFORM TO THE REQUIREMENTS OF ANSI N18.7; (2) PROCEDURES HAD NOT BEEN ISSUED FOR THE INSERVICE TESTING OF PUMPS; AND (3) A MAINTENANCE PROCEDURE FOR RELIEF VALVE TESTING DID NOT CONTAIN APPROPRIATE ACCEPTANCE CRITERIA.
(7901 INFRACTION)

IN DEVIATION FROM THE GUIDANCE OF SECTION 4.2.1 OF ANSI N 18.7-1972 TO WHICH THE LICENSEE WAS COMMITTED BY APPENDIX D OF THE FSAR, THE NUCLEAR AUDIT AND REVIEW COMMITTEE (NARC) CHARTER DID NOT DESCRIBE THE MECHANISM USED TO INITIATE THE NARC AUDITS NOR DID IT IDENTIFY THE MANAGEMENT POSITION TO WHICH NARC AUDITORS IN THE QA ORGANIZATION REPORT WHEN THEY ARE PERFORMING NARC AUDITS.
(7901 DEVIATION)

TECHNICAL SPECIFICATION 6.9.2.B(2) REQUIRES THAT CONDITIONS LEADING TO OPERATION IN A DEGRADED MODE PERMITTED BY A LIMITING CONDITION FOR OPERATION SHALL BE REPORTED TO THE NRC WITHIN 30 DAYS OF OCCURRENCE OF THE EVENT. CONTRARY TO THIS REQUIREMENT, AN OCCURRENCE INVOLVING THE FAILURE OF THE LOOP C TAVE CHANNEL ON OCTOBER 11, 1979, RENDERING THE CORRESPONDING VARIABLE LOW PRESSURE SAFETY INJECTION TRIP CHANNEL INOPERABLE HAD NOT BEEN REPORTED TO THE NRC AT THE TIME OF THE INSPECTION. THE VARIABLE LOW PRESSURE SAFETY INJECTION TRIP CHANNELS ARE REQUIRED TO BE OPERABLE IN ACCORDANCE WITH THE LIMITING CONDITIONS FOR OPERATION PROVISION IN TECHNICAL SPECIFICATION 3.5.1.B.
(7917 DEFICIENCY)

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-25-L0	10-11-79	01-11-80	30 Day LER	Failure of C loop T-ave; disabling low pressure scram channel.
79-26-L0	12-26-79	01-10-80	30 Day LER	Bearing failure in auxiliary feedwater loop.
80-01-T0	01-10-80	01-25-80	14 Day LER	Supports on letdown and turbine plant cooling water lines inadequate.
80-02-T0	01-16-80	01-28-80	14 Day LER	Reactor trip - construction worker bumped protection system relay.

==

INSPECTION STATUS

OTHER ITEMS OF INTEREST (CITE RO REPORT NUMBER, IF APP.)

SYSTEMS AND COMPONENT PROBLEMS:

None.

FACILITY ITEMS (PLANS AND PROCEDURES):

None.

MANAGERIAL ITEMS

None.

PLANT STATUS:

Plant shutdown for TMI modifications.

LAST IE SITE INSPECTION DATE: 01/28-02/07/80+

INSPECTION REPORT NO: 50-206/80-03+

SUMMARY

INSPECTION SUMMARY

+ Inspection on January 7-11, 1980 (Report No. 50-206/80-01)
Report being prepared; to be reported next month.

+ Inspection on January 7-25, 1980 (Report No. 50-206/80-02)
Report being prepared; to be reported next month.

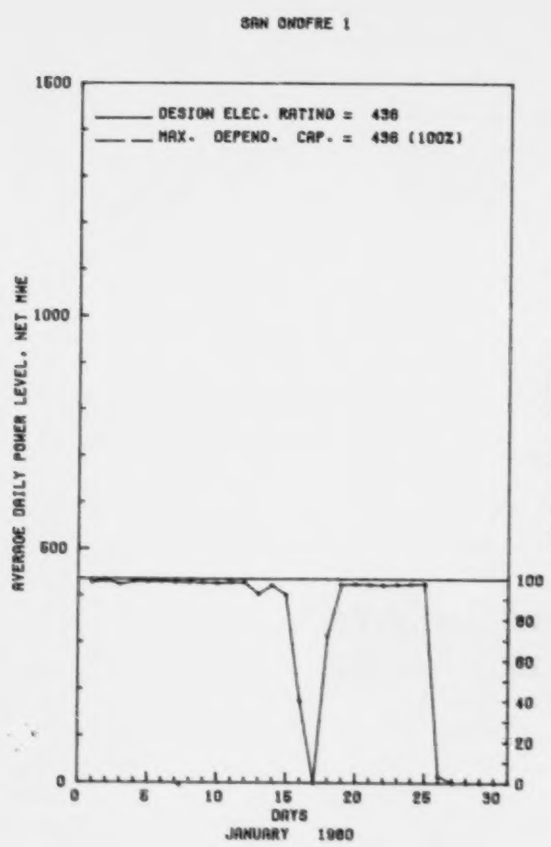
+ Inspection on January 28 - February 7, 1980
(Report No. 50-206/80-03)
Report being prepared; to be reported next month.

REVIEWED BY OMPA LOEB: T. C. Cintula **DATE:** 3•19•80

REVIEWED BY NRR DOR: S. J. Nowicki **DATE:** 3-21-80

SAN ONOFRE 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-206
2. Utility Contact: R. G. Moran (714) 492-7700
3. Licensed Thermal Power (MWt): 1347
4. Nameplate Rating (Gross MWe): 500 x 0.9 = 450
5. Design Electrical Rating (Net MWe): 436
6. Maximum Dependable Capacity (Gross MWe): 456
7. Maximum Dependable Capacity (Net MWe): 436
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	744.0	110,720.0
12. Hours Reactor Was Critical	568.1	568.1	83,200.3
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	564.9	564.9	79,717.3
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	750,876	750,876	102,282,350
17. Gross Electrical Energy (MWH)	247,200	247,200	34,918,634
18. Net Electrical Energy (MWH)	236,191	236,191	33,083,557
19. Unit Service Factor	75.9	75.9	75.5
20. Unit Availability Factor	75.9	75.9	75.5
21. Unit Capacity Factor (MDC Net)	72.8	72.8	72.7
22. Unit Capacity Factor (DER Net)	72.8	72.8	72.7
23. Unit Forced Outage Rate	6.3	6.3	9.3

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUELING - 4/11/80 (7 TO 10 WEEKS)
25. If Shut Down At End of Report Period, Estimated Date of Startup: 02/10/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
39	01/03/80	S	1.3	B	5		HB	VALVEX	TURBINE STOP VALVE TEST.
40	01/13/80	S	5.2	B	5		HB	VALVEX	TURBINE STOP VALVE TEST.
41	01/15/80	S	8.0	B	5		CA	PIPEXX	RCS FLOW TESTS, CLEAR CONDENSER WATER BOXES.
42	01/16/80	F	37.8	H	3	80-002	CH	RELAYX	ACCIDENT CAUSED BY CONSTRUCTION WORKER.
43	01/26/80	S	141.3	H	1		ZZ	ZZZZZZ	TMI MODIFICATIONS.

SUMMARY

OPERATION WAS ROUTINE UNTIL 01/26/80 WHEN THE UNIT WAS TAKEN OFF LINE TO COMPLETE TMI-RELATED MODIFICATIONS. LOAD REDUCTIONS OCCURRED ON 01/03/80, 01/13/80, AND 01/15/80 FOR PERIODIC TESTING. A UNIT TRIP OCCURRED ON 01/16/80 FROM A STEAM FLOW/FEEDWATER FLOW MISMATCH TRIP CAUSED BY A CONSTRUCTION WORKER WHO ACCIDENTALLY STRUCK THE CLOSING CIRCUIT CONTROL RELAY TO THE EAST FEEDWATER PUMP NORMAL DISCHARGE VALVE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER File (NUREG-0161)

[5] Exhibit H: Same source.

ST. LUCIE 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Florida/St. Lucie 12 miles SE of Ft. Pierce
2.	Electric Reliability Council	Southern
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2560
5.	Design Electrical Rating (MWE Net	802
6.	Date of Initial Criticality	April 22, 1976
7.	Date of Commercial Operation	December 21, 1976
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Atlantic Ocean

Utility & Contractor Information

10.	Licensee	Florida Power & Light Co.
11.	Corporate Address	9250 W. Flagler Street P. O. Box 529100 Miami, Florida 33152
12.	Corporate Contact	R. E. Uhrig, V.P.
13.	Architect/Engineer	Ebasco
14.	Nuclear Steam Supply System	Combustion Engineering
15.	Constructor	Ebasco

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. D. MARTIN
18.	IE Senior Inspector	E.H. Verdery
19.	IE Principal Inspector	S. A. ELROD
20.	Licensing Project Manager	P. B. Erickson
21.	Docket Number	50-335
22.	License Number/Date of Issue	DPR-67, March 1, 1976
23.	Public Document Room	Indian River Junior College Library 3209 Virginia Avenue Ft. Pierce, Florida 33450

S-6

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO PART 21 REQUIREMENTS, FPL CORPORATE QA MANUAL QA PROCEDURE, QP 16.4, PARAGRAPH 5.2 AND FIGURE 16.4-1 DOES NOT REQUIRE THAT ALL DEVIATIONS BE FORMALLY EVALUATED AND DOCUMENTED AS PART 21 EVALUATIONS. SINCE FORMAL PART 21 EVALUATIONS ARE NOT PERFORMED AND DOCUMENTED IN ALL CASES, IT FOLLOWS THAT THE INFORMING OF THE RESPONSIBLE OFFICER (2121(A)(2)), THE WRITTEN REPORTS TO THE COMMISSION CONTAIN THE REQUIRED INFORMATION (21.21(B)(3)), AND THE REQUIRED RECORDS BE MAINTAINED (21.51) REQUIREMENTS CANNOT BE MET.
(7933 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

UNIT OPERATING AT 100% POWER.

+ LAST IE SITE INSPECTION DATE: FEBRUARY 1, 1980

+ INSPECTION REPORT NO: 50-335/80-04

SUMMARY

INSPECTION SUMMARY

+ INSPECTION DECEMBER 10-13 (79-32): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 22 INSPECTOR-HOURS ON SITE IN THE AREAS OF PREVIOUSLY IDENTIFIED NONCOMPLIANCES AND OPEN ITEMS, IE BULLETINS, CIRCULARS AND NOTICES AND LICENSEE EVENT REPORTS. OF THE SIX AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

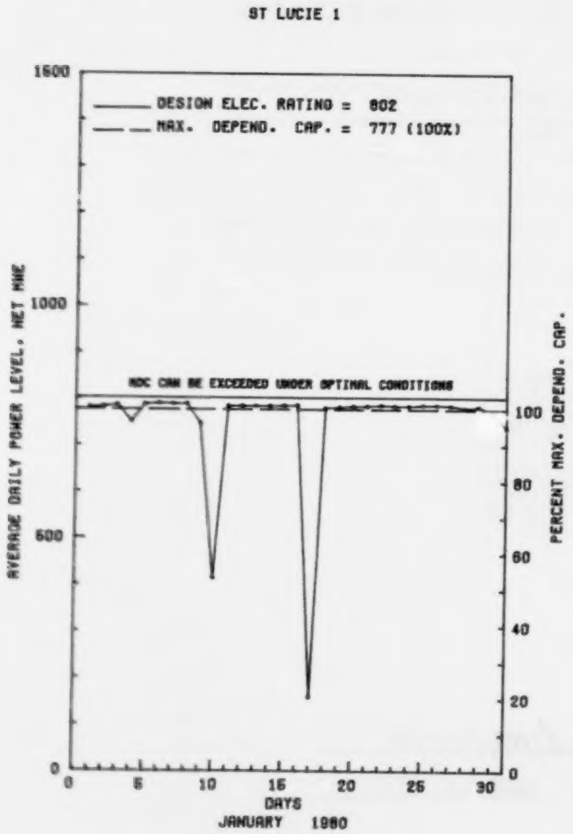
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-05/ 03L-0	12/10/79	01/08/80	30 DAY LER	DISAPPEARANCE OF TEMPERATURE RECORDING THERMOGRAPH FLOTATION BOUY ASSEMBLY.
79-033/ 03L-0	11/19/79	12/18/79	30 DAY LER	DEGRADATION OF FIRE BARRIERS IN REACTOR AUXILIARY BUILDING.
79-034/ 03L-0	11/23/79	12/26/79	30 DAY LER	MB CONTAINMENT ISOLATION SIGNAL CHANNEL FAILURE.
79-035/ 03L-0	11/26/79	12/26/79	30 DAY LER	REFUELING WATER TANK LEVEL CHANNEL C OUT OF TOLERANCE.
79-036/ 03L-0	12/03/79	01/02/80	30 DAY LER	POWER RANGE CHANNEL MB INOPERABLE DUE TO FAILED POWER SUPPLY.

REVIEWED BY OMPA LOEB: MR Beebe DATE: 03/19/80

REVIEWED BY NRR DOR: P B Erickson DATE: 3/20/80

ST. LUCIE 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-335
2. Utility Contact: V. T. Chilson (305) 552-3824
3. Licensed Thermal Power (MWt): 2560
4. Nameplate Rating (Gross MWe): 1000 x 0.85 = 850
5. Design Electrical Rating (Net MWe): 802
6. Maximum Dependable Capacity (Gross MWe): 822
7. Maximum Dependable Capacity (Net MWe): 777
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	744.0	27,288.0
12. Hours Reactor Was Critical	730.4	730.4	22,240.3
13. Reactor Reserve Shutdown Hours	.0	.0	129.4
14. Hours Generator On-Line	727.9	727.9	21,558.0
15. Unit Reserve Shutdown Hours	.0	.0	32.0
16. Gross Thermal Energy (MWH)	1,818,926	1,818,926	52,078,492
17. Gross Electrical Energy (MWH)	589,170	589,170	16,883,670
18. Net Electrical Energy (MWH)	556,641	556,641	15,884,537
19. Unit Service Factor	97.8	97.8	79.0
20. Unit Availability Factor	97.8	97.8	79.1
21. Unit Capacity Factor (MDC Net)	96.3	96.3	74.9
22. Unit Capacity Factor (DER Net)	93.3	93.3	72.6
23. Unit Forced Outage Rate	2.2	2.2	5.3

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUELING, MAINTENANCE, & INSPECTIONS - 03/16 - 05/13/80.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
1	01/09/80	S	.0	B	5		HH	FILTER	LOAD REDUCTION TO CLEAN SUCTION STRAINERS ON CONDENSATE PUMPS AND STEAM GENERATOR FEEDWATER PUMPS. (NON-NUCLEAR SYSTEM).
2	01/17/80	F	16.1	A	3		IA	INSTRU	UNIT WAS TRIPPED BY THE REACTOR PROTECTION SYSTEM DURING A PERIODIC SURVEILLANCE TEST WHEN THE SECOND REACTOR TRIP BREAKER, IN A TRIP-PATH, FAILED TO REMAIN CLOSED AFTER TESTING. OUTAGE WAS EXTENDED TO REPLACE A FAILED CABLE IN THE POWER SUPPLY TO CEDM NO. 1.
3	01/30/80	F	.0	A	5		RB	ELECON	LOAD REDUCTION TO RETRIEVE AND REALIGN CEA NO. 41. CORRECTIVE ACTIONS INCLUDED REPAIRING A FAULTY ELECTRICAL CONNECTION.

SUMMARY

OPERATION WAS AT NEAR FULL POWER DURING THE REPORT PERIOD, WITH A UNIT TRIP ON 01/17/80 BY THE REACTOR PROTECTION SYSTEM DURING A PERIODIC SURVEILLANCE TEST WHEN THE SECOND REACTOR TRIP BREAKER FAILED TO REMAIN CLOSED AFTER TESTING. LOAD REDUCTIONS OCCURRED ON 01/09/80 TO CLEAN SUCTION STRAINERS ON CONDENSATE AND STEAM GENERATOR FEEDWATER PUMPS, AND ON 01/30/80 TO RETRIEVE AND REALIGN CONTROL ELEMENT ASSEMBLY NO. 41.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

SURRY 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Virginia/Surry 17 miles NW of Newport News
2.	Electric Reliability Council	Southeastern
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2441
5.	Design Electrical Rating (MWE Net)	822
6.	Date of Initial Criticality	July 1, 1972
7.	Date of Commercial Operation	December 22, 1972
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	James River

Utility & Contractor Information

10.	Licensee	Virginia Electric and Power Company
11.	Corporate Address	P. O. Box 26666 Richmond, Virginia 23261
12.	Corporate Contact	J. H. Ferguson Exec. V.P. - Power
13.	Architect/Engineer	Stone & Webster
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R.C. Lewis
18.	IE Senior Inspector	P. J. Kellogg
19.	IE Resident Inspector	D. Burke
20.	Licensing Project Manager	D. Neighbors
21.	Docket Number	50-280
22.	License Number/Date of Issue	DPR-32, 5/25/72
23.	Public Document Room	Swem Library College of William and Mary Williamsburg, Virginia 23185

INSPECTION STATUS

ENFORCEMENT STATUS

AS REQUIRED BY TECHNICAL SPECIFICATION 6.4.B.1.6, AREAS WITH RADIATION LEVELS IN EXCESS OF 1000 MREM/HR SHALL BE PROVIDED WITH LOCKED BARRICADES TO PREVENT UNAUTHORIZED ENTRY INTO THESE AREAS. CONTRARY TO THE ABOVE, ON NOVEMBER 27, 1979, THE GATE CONTROLLING ACCESS TO THE BORIC ACID FILTER AREA (GATE NO. 7) DID NOT HAVE A LOCK ADEQUATE TO PREVENT UNAUTHORIZED ENTRY. RADIATION LEVELS WERE 2000 MREM/HR. (7964 INFRACTION)

CONTRARY TO THE 10 CFR 50, APPENDIX B, CRITERION V, SECTION 1-6 OF PT'S 2.1-2.11 GENERATED BETWEEN 1/1/79 AND 11/27/79 WERE NOT REVIEWED BY THE COGNIZANT SUPERVISOR AND PERFORMANCE ENGINEER, INITIALED AND SIGNED IN INK, AND RETAINED IN THE STATION FILE ROOM FOR FIVE YEARS. (7965 DEFICIENCY)

CONTRARY TO THE TECHNICAL SPECIFICATION 6.4.D, THE AUXILIARY FEED PUMP DISCHARGE VALVES (MOV-FW-151C AND MOV-FW-151E) WERE TESTED ON 10/13/79 AND THE OPENING TIME EXCEEDED THE 25% LIMIT. NO STATION DEVIATION WAS SUBMITTED FOR THESE VALVES AND THE ERROR WAS NOT IDENTIFIED IN THE REVIEW PROCESS. (7965 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

RE-ANALYSIS AND REVIEW OF SEISMIC DESIGN DEFICIENCIES ARE CONTINUING.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

UNIT IS CURRENTLY OPERATING.

+ LAST IE SITE INSPECTION DATE: FEBRUARY 1, 1980

+ INSPECTION REPORT NO: 50-280/80-02

SUMMARY

INSPECTION SUMMARY

+ INSPECTION NOVEMBER 27-30 (79-64): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 34 INSPECTOR-HOURS ON SITE IN THE AREAS OF SHIPMENTS OF RADIOACTIVE MATERIALS AND FACILITY TOURS. OF THE TWO AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN ONE AREA; ONE APPARENT ITEM OF NONCOMPLIANCE WAS FOUND IN ONE AREA (INFRACTION - HIGH RADIATION AREA LOCK INADEQUATE TO PREVENT UNAUTHORIZED ACCESS).

INSPECTION NOVEMBER 26-30 (79-65): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 33 INSPECTOR-HOURS ON SITE IN THE AREAS OF CALIBRATION OF IN-PLANT INSTRUMENTATION REQUIRED BY TECHNICAL SPECIFICATIONS; CALIBRATION OF MEASURING AND TEST EQUIPMENT USED ON SAFETY-RELATED SYSTEMS; AND SURVEILLANCE OF IN-PLANT EQUIPMENT REQUIRED BY TECHNICAL SPECIFICATIONS. OF THE THREE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN ONE AREA; TWO APPARENT ITEMS OF NONCOMPLIANCE WERE FOUND IN TWO AREAS (INFRACTION - FAILURE TO FOLLOW SURVEILLANCE TEST PROCEDURE; AND DEFICIENCY - FAILURE TO REVIEW AND RETAIN INSTRUMENT CALIBRATION PROCEDURES).

INSPECTION DECEMBER 3-7 (79-66): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 28 INSPECTOR-HOURS ON SITE IN THE AREAS OF RADIOLOGICAL ENVIRONMENTAL MONITORING PROGRAM INCLUDING: MANAGEMENT CONTROL, QUALITY CONTROL OF ANALYTICAL MEASUREMENTS, INSPECTION OF ENVIRONMENTAL MONITORING STATIONS, REVIEW OF ENVIRONMENTAL MONITORING DATA, REVIEW OF RADIOLOGICAL ENVIRONMENTAL MONITORING PROCEDURES, AND IMPLEMENTATION OF THE MONITORING PROGRAM. OF THE SIX AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION NOVEMBER 1 - DECEMBER 14 (79-67): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 120 INSPECTOR-HOURS ON SITE IN THE AREAS OF PLANT OPERATIONS AND OPERATING RECORDS, PLANT MAINTENANCE, AND PLANT SECURITY. OF THE THREE AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE FOUND IN TWO AREAS; ONE APPARENT ITEM OF NONCOMPLIANCE WAS FOUND IN THE REMAINING AREA (INFRACTION - FAILURE TO FOLLOW CORRECTIVE MAINTENANCE PROCEDURES).

INSPECTION DECEMBER 10-12 (79-68): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 40 INSPECTOR-HOURS ON SITE IN THE AREAS OF CONSTRUCTION STATUS, ENGINEERING TASK ASSIGNMENT REVIEW, LICENSEE ACTION ON PREVIOUS INSPECTION FINDINGS, NONCONFORMANCE REPORT REVIEW, ON SITE DRAWING/ETA REVIEW, AND LONG TERM STORAGE OF STEAM GENERATOR LOWER ASSEMBLIES. OF THE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

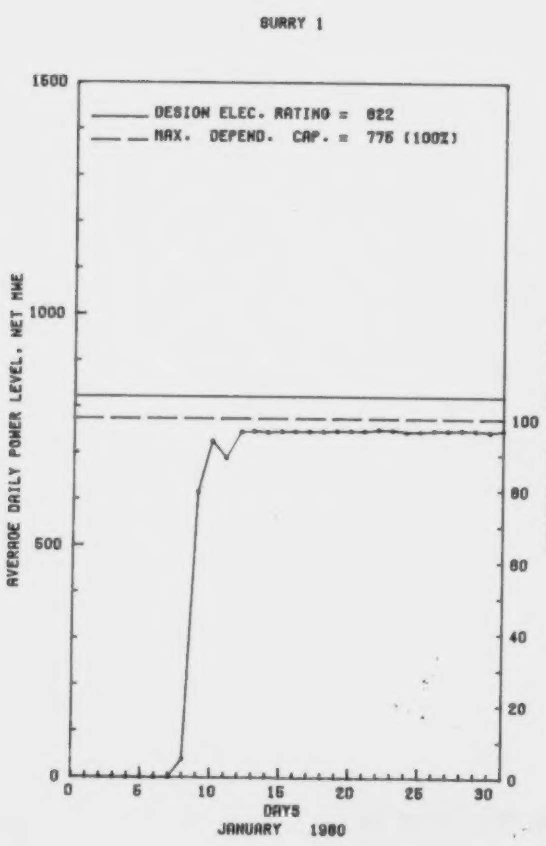
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-033/ 03L-0	11/27/79	12/20/79	30 DAY LER	SERVICE WATER TO CHARGING PUMPS INOPERABLE.
79-034/ 01T-0	12/07/79	12/20/79	14 DAY LER	CONTROL CIRCUITRY FOR CONDENSER AIR EJECTOR DISCHARGE ISOLATION VALVES TO THE CONTAINMENT NOT IN ACCORDANCE WITH FSAR.
79-035/ 03L-0	11/27/79	12/20/79	30 DAY LER	HEAT TRACING CIRCUIT FAILURE.
79-038/ 03L-0	12/14/79	01/03/80	30 DAY LER	TWO MANUAL VALVES ON CHEMICAL ADDITION TANK FOUND OUT OF POSITION DURING PLANT OPERATION.
79-039/ 03L-0	12/14/79	01/10/80	30 DAY LER	FAILURE OF HEAT TRACING CIRCUIT ON BORIC ACID TRANSFER PUMP STRIP HEATER.
79-040/ 01T-0	12/18/79	12/27/79	14 DAY LER	TWO COMPONENT COOLING WATER PIPE SUPPORTS OVERSTRESSED.
79-041/ 21X-0	12/19/79	01/03/80	14 DAY LER	REACTOR TRIP AND SAFETY INJECTION ON LOSS OF A REACTOR COOLANT PUMP.
79-042/ 01T-0	12/20/79	01/02/80	14 DAY LER	OVERSTRESS ON COMPONENT COOLING WATER SYSTEM PIPE HANGER.
79-045/ 01T-0	12/28/79	01/10/80	14 DAY LER	DESIGN OF SAFEGUARDS AREA EXHAUST VENTILATION DAMPERS IS NOT IN ACCORDANCE WITH FSAR.

REVIEWED BY OMPA LOEB: DATE: 3/20/80

REVIEWED BY NRR DOR: Don Neighbors DATE: 3/20/80

SURRY 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-280
2. Utility Contact: O. J. Costello (804) 357-3184
3. Licensed Thermal Power (MWt): 2441
4. Nameplate Rating (Gross MWe): 942 x 0.9 = 848
5. Design Electrical Rating (Net MWe): 822
6. Maximum Dependable Capacity (Gross MWe): 811
7. Maximum Dependable Capacity (Net MWe): 775
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	744.0	62,328.0
12. Hours Reactor Was Critical	561.4	561.4	39,313.0
13. Reactor Reserve Shutdown Hours	182.6	182.6	3,731.5
14. Hours Generator On-Line	556.1	556.1	38,461.1
15. Unit Reserve Shutdown Hours	183.6	187.9	3,740.5
16. Gross Thermal Energy (MWH)	1,337,473	1,337,473	89,052,345
17. Gross Electrical Energy (MWH)	430,185	430,185	29,110,683
18. Net Electrical Energy (MWH)	409,019	409,019	27,635,918
19. Unit Service Factor	74.7	74.7	61.7
20. Unit Availability Factor	99.4	99.4	67.7
21. Unit Capacity Factor (MDC Net)	70.9	70.9	57.2
22. Unit Capacity Factor (DER Net)	66.9	66.9	53.9
23. Unit Forced Outage Rate	25.3	25.3	24.8

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUELING JUNE 1980 - 6 WEEKS
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-1	01/01/80	F	183.6	D	9				UNIT DOWN FOR REPLACEMENT OF RCP MOTOR. NRC REQUIREMENT TO SUCCESSFULLY TEST RCP SNUBBER PRIOR TO STARTUP.
80-2	01/08/80	F	2.5	G	3				REACTOR TRIP CAUSED BY LO-LO LEVEL IN "A" STEAM GENERATOR FEEDING S/G'S IN MANUAL.
80-3	01/08/80	F	1.8	G	3				REACTOR TRIP CAUSED BY HIGH LEVEL IN "A" S/G WHILE FEEDING S/G'S IN MANUAL.

SUMMARY

OPERATION WAS ROUTINE FOLLOWING RETURN TO POWER ON 01/08/80 FROM AN OUTAGE TO REPLACE THE REACTOR COOLANT PUMP MOTOR. TWO REACTOR TRIPS ALSO OCCURRED ON 01/08/80, ONE FROM LO-LO LEVEL AND ONE FROM HIGH LEVEL IN THE "A" STEAM GENERATOR.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

SURRY 2

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Virginia/Surry 17 miles NW of Newport News
2.	Electric Reliability Council	Southeastern
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2441
5.	Design Electrical Rating (MWE Net)	822
6.	Date of Initial Criticality	March 7, 1973
7.	Date of Commercial Operation	May 1, 1973
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	James River

Utility & Contractor Information

10.	Licensee	Virginia Electric and Power Company
11.	Corporate Address	P. O. Box 26666 Richmond, Virginia 23261
12.	Corporate Contact	J. H. Ferguson Exec. V.P. - Power
13.	Architect/Engineer	Stone & Webster
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R.C. Lewis
18.	IE Senior Inspector	P. J. Kellog
19.	IE Resident Inspector	D. Burke
20.	Licensing Project Manager	D. Neighbors
21.	Docket Number	50-281
22.	License Number/Date of Issue	DPR-37, 1/29/73
23.	Public Document Room	Swem Library College of William and Mary Williamsburg, Virginia 23185

INSPECTION STATUS

ENFORCEMENT STATUS

AS REQUIRED BY TECHNICAL SPECIFICATION 6.4.B.1.6, AREAS WITH RADIATION LEVELS IN EXCESS OF 1000 MREM/HR SHALL BE PROVIDED WITH LOCKED BARRICADES TO PREVENT UNAUTHORIZED ENTRY INTO THESE AREAS. CONTRARY TO THE ABOVE, ON NOVEMBER 27, 1979, THE GATE CONTROLLING ACCESS TO THE BORIC ACID FILTER AREA (GATE NO. 7) DID NOT HAVE A LOCK ADEQUATE TO PREVENT UNAUTHORIZED ENTRY. RADIATION LEVELS WERE 2000 MREM/HR. (7984 INFRACTION)

CONTRARY TO THE 10 CFR 50, APPENDIX B, CRITERION V, SECTION 1-6 OF PT'S 2.1-2.11 GENERATED BETWEEN 1/1/79 AND 11/27/79 WERE NOT REVIEWED BY THE COGNIZANT SUPERVISOR AND PERFORMANCE ENGINEER, INITIALED AND SIGNED IN INK, AND RETAINED IN THE STATION FILE ROOM FOR FIVE YEARS. (7985 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

SYSTEM MODIFICATIONS FOR SEISMIC SUPPORT REANALYSIS AND NUREG 0578 ARE CONTINUING.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

UNIT IS CURRENTLY SHUT DOWN AND DEFUELED. THE STEAM GENERATOR REPLACEMENT HAS BEEN COMPLETED.

LAST IE SITE INSPECTION DATE: FEBRUARY 1, 1980

INSPECTION REPORT NO: 50-281/80-04

SUMMARY

INSPECTION SUMMARY

INSPECTION NOVEMBER 27-30 (79-84): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 34 INSPECTOR-HOURS ON SITE IN THE AREAS OF SHIPMENTS OF RADIOACTIVE MATERIALS AND FACILITY TOURS. OF THE TWO AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN ONE AREA; ONE APPARENT ITEM OF NONCOMPLIANCE WAS FOUND IN ONE AREA (INFRACTION - HIGH RADIATION AREA LOCK INADEQUATE TO PREVENT UNAUTHORIZED ACCESS).

INSPECTION NOVEMBER 26-30 (79-85): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 33 INSPECTOR-HOURS ON SITE IN THE AREAS OF CALIBRATION OF IN-PLANT INSTRUMENTATION REQUIRED BY TECHNICAL SPECIFICATIONS; CALIBRATION OF MEASURING AND TEST EQUIPMENT USED ON SAFETY-RELATED SYSTEMS; AND SURVEILLANCE OF IN-PLANT EQUIPMENT REQUIRED BY TECHNICAL SPECIFICATIONS. OF THE THREE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN ONE AREA; TWO APPARENT ITEMS OF NONCOMPLIANCE WERE FOUND IN TWO AREAS (INFRACTION - FAILURE TO FOLLOW SURVEILLANCE TEST PROCEDURE; AND DEFICIENCY - FAILURE TO REVIEW AND RETAIN INSTRUMENT CALIBRATION PROCEDURES).

INSPECTION DECEMBER 3-7 (79-86): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 28 INSPECTOR-HOURS ON SITE IN THE AREAS OF RADIOLOGICAL ENVIRONMENTAL MONITORING PROGRAM INCLUDING: MANAGEMENT CONTROL, QUALITY CONTROL OF ANALYTICAL MEASUREMENTS, INSPECTION OF ENVIRONMENTAL MONITORING STATIONS, REVIEW OF ENVIRONMENTAL MONITORING DATA, REVIEW OF RADIOLOGICAL ENVIRONMENTAL MONITORING PROCEDURES, AND IMPLEMENTATION OF THE MONITORING PROGRAM. OF THE SIX AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
None				

INSPECTION DECEMBER 10-12 (79-87): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 40 INSPECTOR-HOURS ON SITE IN THE AREAS OF CONSTRUCTION STATUS, ENGINEERING TASK ASSIGNMENT REVIEW, LICENSEE ACTION ON PREVIOUS INSPECTION FINDINGS, NONCONFORMANCE REPORT REVIEW, ON SITE DRAWING/ETA REVIEW, AND LONG TERM STORAGE OF STEAM GENERATOR LOWER ASSEMBLIES. OF THE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION DECEMBER 12-14 (79-88): THIS SPECIAL, UNANNOUNCED INSPECTION INVOLVED 15 INSPECTOR-HOURS ON SITE IN THE AREA OF PIPE SUPPORT BASE PLATE DESIGNS USING CONCRETE EXPANSION ANCHOR BOLTS - IE BULLETIN (IEB) 79-02. NO ITEMS OF NONCOMPLIANCE OF DEVIATIONS WERE IDENTIFIED.

INSPECTION NOVEMBER 1 - DECEMBER 14 (79-89): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 120 INSPECTOR-HOURS ON SITE IN THE AREAS OF PLANT OPERATIONS AND OPERATING RECORDS, PLANT MAINTENANCE, AND PLANT SECURITY. OF THE THREE AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE FOUND IN TWO AREAS; ONE APPARENT ITEM OF NONCOMPLIANCE WAS FOUND IN THE REMAINING AREA (INFRACTION - FAILURE TO FOLLOW CORRECTIVE MAINTENANCE PROCEDURES).

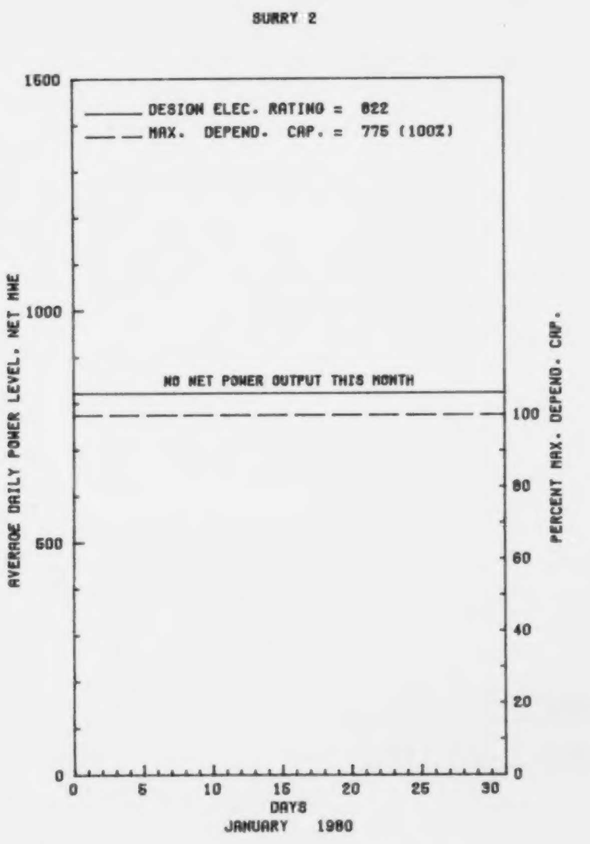
INSPECTION JANUARY 14-17 (80-02): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 28 INSPECTOR-HOURS ON SITE IN THE AREAS OF WELDING FILLER MATERIAL CONTROL, AND NONDESTRUCTIVE EXAMINATION. OF THE TWO AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

REVIEWED BY OMPA LOEB: **DATE:** 3/20/80

REVIEWED BY NRR DOR: Don Neighbors **DATE:** 3/20/80

SURRY 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-281
2. Utility Contact: C. J. Costello (804) 357-3184
3. Licensed Thermal Power (MWt): 2441
4. Nameplate Rating (Gross MWe): 942 x 0.9 = 848
5. Design Electrical Rating (Net MWe): 822
6. Maximum Dependable Capacity (Gross MWe): 811
7. Maximum Dependable Capacity (Net MWe): 775
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	744.0	59,208.0
12. Hours Reactor Was Critical	.0	.0	34,499.9
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	.0	.0	33,996.2
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	0	79,194,083
17. Gross Electrical Energy (MWH)	0	0	25,868,844
18. Net Electrical Energy (MWH)	0	0	24,536,605
19. Unit Service Factor	.0	.0	57.4
20. Unit Availability Factor	.0	.0	57.4
21. Unit Capacity Factor (MDC Net)	.0	.0	53.5
22. Unit Capacity Factor (DER Net)	.0	.0	50.4
23. Unit Forced Outage Rate	.0	.0	21.0

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 05/05/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-1	02/04/79	S	744.0	C	4				CONTINUATION OF SHUTDOWN FOR REFUELING AND STEAM GENERATOR REPLACEMENT WHICH BEGAN ON 2/4/79.

SUMMARY

THE UNIT REMAINED SHUTDOWN THROUGHOUT THE REPORT PERIOD. HYDROSTATIC TESTS ON THE REPLACEMENT STEAM GENERATORS WERE COMPLETED THIS MONTH.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

THREE MILE ISLAND 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Pennsylvania/Dauphin 10 miles SE of Harrisburg
2.	Electric Reliability Council	MAAC
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2535
5.	Design Electrical Rating (MWE Net)	819
6.	Date of Initial Criticality	June 5, 1974
7.	Date of Commercial Operation	September 2, 1974
8.	Condenser Cooling Method	Closed
9.	Condenser Cooling Water Source	Susquehanna River

Utility & Contractor Information

10.	Licensee	Metropolitan Edison Company
11.	Corporate Address	P. O. Box 542 Reading, Pennsylvania
12.	Corporate Contact	R. C. Arnold, Sr. Vice-President 260 Cherry Hill Rd. Parsippany, NJ 07054
13.	Architect/Engineer	Gilbert Associates, Inc.
14.	Nuclear Steam Supply System	B&W
15.	Constructor	UE&C

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Senior Inspector	R. R. Keimig
19.	IE Principal Inspector	D. R. Haverkamp
20.	Licensing Project Manager	D. C. Dilanni
21.	Docket Number	50-289
22.	License Number/Date of Issue	DPR-50, 4/19/74
23.	Public Document Room	Government Publications Section State Library of Pennsylvania Education Building Commonwealth and Walnut Street Harrisburg, Pennsylvania 17126

INSPECTION STATUS

ENFORCEMENT STATUS

LICENSEE VEHICLE NOT ESCORTED AS REQUIRED.
(7919 INFRACTION)

VITAL AREA DOOR LOCKS NOT MAINTAINED IN OPERABLE CONDITION.
(7919 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

None

FACILITY ITEMS (PLANS AND PROCEDURES):

None

MANAGERIAL ITEMS:

* Mr. R. M. Toole was appointed to the new position of TMI-1 Manager effective February 1, 1980.

PLANT STATUS:

The plant remains shutdown by NRC order, pending completion of modifications and other actions related to the TMI-2 accident.

* LAST IE SITE INSPECTION DATE: 2/20/80

* INSPECTION REPORT NO: 50-289/80-02

SUMMARY

INSPECTION SUMMARY

* No inspection reports issued.

REPORTS RECEIVED FROM LICENSEE

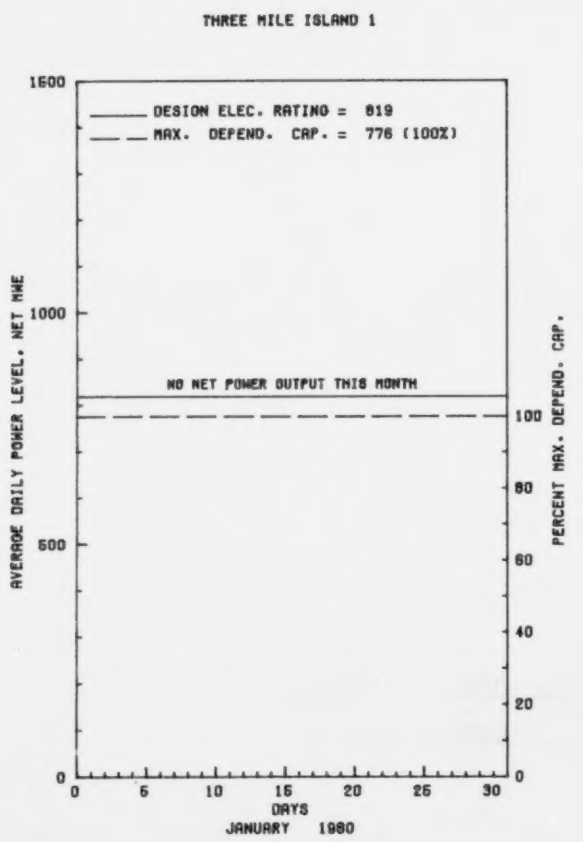
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-017/ 01T	11/08/79	12/11/79	14 Day LER	Inadequate bolt installation and design in rigid support anchor for decay heat removal cooling piping

REVIEWED BY OMPA LOEB: John J. [illegible] **DATE:** 3-19-80

REVIEWED BY NRR DOR: D C Di[illegible] **DATE:** 3-20-80

THREE MILE ISLAND 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-289
2. Utility Contact: D. G. Mitchell (215) 921-6579
3. Licensed Thermal Power (MWt): 2535
4. Nameplate Rating (Gross MWe): 968 x 0.9 = 871
5. Design Electrical Rating (Net MWe): 819
6. Maximum Dependable Capacity (Gross MWe): 840
7. Maximum Dependable Capacity (Net MWe): 776
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	744.0	47,473.0
12. Hours Reactor Was Critical	.0	.0	31,731.8
13. Reactor Reserve Shutdown Hours	.0	.0	838.5
14. Hours Generator On-Line	.0	.0	31,130.9
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	0	76,531,071
17. Gross Electrical Energy (MWH)	0	0	25,484,330
18. Net Electrical Energy (MWH)	0	0	23,840,053
19. Unit Service Factor	.0	.0	65.7
20. Unit Availability Factor	.0	.0	65.7
21. Unit Capacity Factor (MDC Net)	.0	.0	64.7
22. Unit Capacity Factor (DER Net)	.0	.0	61.3
23. Unit Forced Outage Rate	100.0	100.0	22.2

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: UNKNOWN

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
1	03/01/79	F	744.0	D	4				REGULATORY RESTRAINT ORDER

SUMMARY

THE UNIT REMAINS SHUTDOWN DUE TO REGULATORY CONSIDERATION OF THE TMI-2 ACCIDENT.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG 0161)

[5] Exhibit H:
Same source.

THREE MILE ISLAND 2

NRC POWER RESTRICTIONS: SEE "LICENSEE STATUS"

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Pennsylvania/Dauphin 10 miles SE of Harrisburg
2.	Electric Reliability Council	MAAC
3.	Type of Reactor	PWR
4.	Licensed Power Level (MWT)	2772
5.	Design Electrical Rating (MWE Net)	906
6.	Date of Initial Criticality	3/28/78
7.	Date of Commercial Operation	12/30/78
8.	Condenser Cooling Method	CLOSED
9.	Condenser Cooling Water Source	Susquehanna River

Utility & Contractor Information

10.	Licensee	Metropolitan Edison Company
11.	Corporate Address	P. O. Box 542 Reading, PA 19603
12.	Corporate Contact	R. C. Arnold, Sr. Vice-President 260 Cherry Hill Rd. Parsippany, NJ 07054
13.	Architect/Engineer	Burns & Roe
14.	Nuclear Steam Supply System	B&W
15.	Constructor	UE&C

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Senior Inspector	R. R. Keimig
19.	IE Principal Inspector	R. Conte FTS 590-3952
20.	Licensing Project Manager	R. VOLLMER
21.	Docket Number	50-320
22.	License Number/Date of Issue	DPR-73/2; 2/8/78
23.	Public Document Room	Government Publications Section State Library of Pennsylvania Education Building Commonwealth & Walnut Streets Harrisburg, Pennsylvania 17126

INSPECTION STATUS

ENFORCEMENT STATUS

LICENSEE VEHICLE NOT ESCORTED AS REQUIRED.
(7925 INFRACTION)

VITAL AREA DOOR LOCKS NOT MAINTAINED IN OPERABLE CONDITION.
(7925 INFRACTION)

10 CFR 20.207, "STORAGE & CONTROL OF LICENSED MATERIALS IN UNRESTRICTED AREAS," STATES, "(A) LICENSED MATERIALS STORED IN AN UNRESTRICTED AREA SHALL BE SECURED FROM UNAUTHORIZED REMOVAL FROM THE PLACE OF STORAGE (B) LICENSED MATERIALS IN AN UNRESTRICTED AREA & NOT IN STORAGE SHALL BE TENDED UNDER THE CONSTANT SURVEILLANCE & IMMEDIATE CONTROL OF THE LICENSEE." CONTRARY TO THE ABOVE, ON 11/15/79, AT LEAST 2 DRUMS OF LICENSED MATERIAL (MISTAKEN TO BE NON-RADIOACTIVE) WERE TRANSFERRED TO A VENDOR FOR TRANSPORT & DISPOSAL AT THE HARRISBURG, PA. CITY REFUGE DUMP. ON 11/16/79, THE 2 DRUMS WERE SURVEYED & FOUND TO HAVE RADIATION LEVELS OF 0.5 & 3.0 MREM/HR, RESPECTIVELY.
(7929 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

Recovery operations from the 3/28/79 incident.

FACILITY ITEMS (PLANS AND PROCEDURES):

None

MANAGERIAL ITEMS:

None

PLANT STATUS:

No significant information to report.

+ LAST IE SITE INSPECTION DATE: 1/31/80

+ INSPECTION REPORT NO: 50-320/80-01

SUMMARY

INSPECTION SUMMARY

+ No inspections completed.

LICENSE STATUS

By Order for Modification of License dated July 20, 1979, the licensee's authority to operate the facility was suspended and the licensee's authority was limited to maintenance of the facility in the present shutdown cooling mode (44 F.R. 45271).

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
	NONE			

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 3-19-80

REVIEWED BY NRR DOR: **DATE:** 3-21-80

THREE MILE ISLAND 2

AVERAGE DAILY POWER LEVEL (MWe)

OPERATING STATUS

1. Docket Number: 50-320
2. Utility Data Prepared By: R. A. Lengel (215) 921-6581
3. Licensed Thermal Power (MWt): 2772
4. Nameplate Rating (Gross MWe): 1068 X 0.9 = 961
5. Design Electrical Rating (Net MWe): 906
6. Maximum Dependable Capacity (Gross MWe): 0
7. Maximum Dependable Capacity (Net MWe): 0
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons: None
9. Power Level To Which Restricted, If Any (Net MWe): None
10. Reasons For Restrictions, If Any:

This Month Yr

11. Hours In Reporting Period
12. Number Of Hours Reactor Was Critical
13. Reactor Reserve Shutdown Hours
14. Hours Generator On-Line
15. Unit Reserve Sh
16. Gro

(MWH)

19

20. U Availability Factor
21. Unit Capacity Factor (Using MDC Net) 1/
22. Unit Capacity Factor (Using DER Net) 1/
23. Unit Forced Outage Rate
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): None
25. If Shut Down At End Of Report Period, Estimated Date of Startup: N/A

NO DECISION ON FUTURE OPERATION
SHUTDOWN INDEFINITELY

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
SEE SUMMARY BELOW.									

SUMMARY

THE UNIT HAS REMAINED SHUTDOWN SINCE THE 03/28/79 ACCIDENT, WHICH CAUSED EXTENSIVE CORE DAMAGE.

AUTHORITY TO OPERATE FACILITY SUSPENDED BY NRC ORDER DATED JULY 20, 1979, WHICH REQUIRES LICENSEE TO MAINTAIN FACILITY IN A SHUTDOWN CONDITION.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

TROJAN

FACILITY DATA

Facility Description

1. Location (state/countv) (nearest population center): Oregon/Columbia 42 miles N of Portland
2. Electric Reliability Council: WSCC
3. Type of Reactor: PWR
4. Licenses Power Level (MWT): 3411
5. Design Electrical Rating (MWE Net): 1130
6. Date of Initial Criticality: December 15, 1975
7. Date of Commercial Operation: May 20, 1976
8. Condenser Cooling Method: Natural draft cooling tower
9. Condenser Cooling Water Source: Columbia River

Utility & Contractor Information

10. Licensee: Portland General Electric Co.
11. Corporate Address: 121 S. W. Salmon St. Portland, Oregon 97204
12. Corporate Contact: C. Goodwin, Jr., Asst. V.P.
13. Architect/Engineer: Bechtel
14. Nuclear Steam Supply System: Westinghouse
15. Constructor: Multiple

Regulatory Information

16. IE Region responsible: V
17. IE Branch Chief: J. L. Crews
18. IE Senior Inspector: D. M. Sternberg
19. IE Principal Inspector (Resident): M. H. Malmros
20. Licensing Project Manager: C. M. Trammell
21. Docket Number: 50-344
22. License Number/Date of Issue: NPF-1, 11/21/75
23. Public Document Room: Columbia County Courthouse Law Library Circuit Court Room St. Helens, Oregon 97501

INSPECTION STATUS

ENFORCEMENT STATUS

FAILURE TO REMOVE EMPLOYEE FROM ACCESS LIST AND REMOVE ACCESS BADGE FROM ENTRY PORTAL UPON TERMINATION OF THE EMPLOYEE'S NEED FOR ACCESS. (7920 INFRACTION)

FAILURE OF GUARD TO CHECK POCKET CARRIED IDENTIFICATION OF VISITOR AFFILIATION PRIOR TO ALLOWING PROTECTED AREA ENTRY. (7920 INFRACTION)

CONTRACT GUARD ALLOWED A CONTRACT CLEANER TO BRING A FRIEND ONSITE AND INTO VITAL AREAS WITHOUT APPROVAL OF PLANT MANAGEMENT. (7920 VIOLATION)

FACILITY OPERATING LICENSE NO. NPF-1 REQUIRES IN SECTION 2.C.(7) THAT "....ALL ACTIVITIES TO WHICH A QUALITY ASSURANCE PROGRAM IS APPLICABLE SHALL, AFTER THE DATE OF ISSUE OF THE LICENSE, BE CONDUCTED IN ACCORDANCE WITH QUALITY ASSURANCE PROGRAM FOR OPERATION." CONTRARY TO THIS REQUIREMENT, THE REQUIREMENTS OF QUALITY ASSURANCE PROCEDURE, QAP 12-1, "INSTALLED INSTRUMENTATION CALIBRATION CONTROL PROGRAM," WERE NOT ADHERED TO IN THAT A TEST GAUGE NOT IN A CURRENT STATE OF CALIBRATION WAS USED DURING THE CALIBRATION OF AN INSTALLED PLANT INSTRUMENT. (7923 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST (CITE RO REPORT NUMBER, IF APP.)

SYSTEMS AND COMPONENT PROBLEMS:

- None.

FACILITY ITEMS (PLANS AND PROCEDURES):

- The Phase Two public hearing on control building modifications is planned for April 1980.

MANAGERIAL ITEMS:

None.

PLANT STATUS

- The plant operated at power throughout the month except for a shutdown on January 6-7, 1980 (manual trip because of low condenser vacuum). At the end of January, the plant was operating at full power.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
--	10-18-79	10-24-79	Environmental	Plant effluent above technical specification limit for pH.
--	09-06-79	12-03-79	Special	Special report on ECCS actuation.
79-16	12-08-79	01-04-80	30 Day LER	Leaking pipe in coolant charging pump miniflow recirculating line.

LAST IE SITE INSPECTION DATE: 01/02-31/80♦

INSPECTION REPORT NO: 50-344/80-01♦

SUMMARY

INSPECTION SUMMARY:

- Inspection on December 1-31, 1979 (Report No. 50-344/79-24) Routine inspections of plant operations, maintenance, physical security, procurement, radiological waste operations, organization, audit and review, and followup on a Licensee Event Report. The inspection involved 75 inspector hours by a Regional Inspector and the NRC Resident Inspector.

  Results: No items of noncompliance or deviations were identified.

- Inspection on January 2-31, 1980 (Report No. 50-344/80-01) Report being prepared; to be reported next month.

- Inspection on January 21-23, 1980 (Report No. 50-344/80-02) Routine, unannounced inspection of quality assurance program, operator requalification program, response to IE Bulletins and Circulars, and followup on previous inspection items, also included independent inspection effort. The inspection involved 32 inspector hours onsite by two NRC inspectors.

  Results: Of the three areas inspected, one noncompliance item (Deficiency - Records of reactivity control manipulations) and two deviations (related to review of operator requalification program) were identified in one area. No items of noncompliance or deviations were identified in the other two areas inspected.

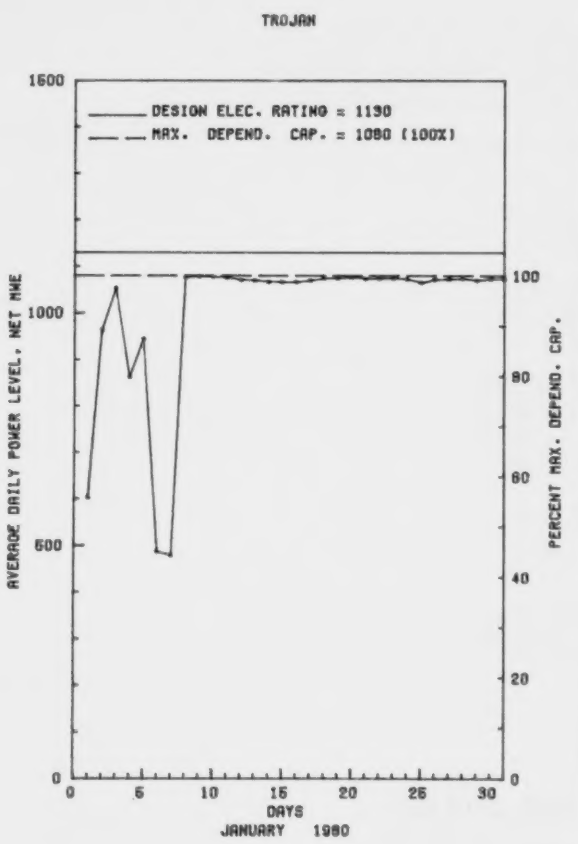
REVIEWED BY OMPA LOEB: **DATE:** 3/20/80

REVIEWED BY NRR DOR: CM Tramell **DATE:** 3/20/80

GPO 871-667

TROJAN

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-344
2. Utility Contact: G. G. Rair (503) 556-3713 x234
3. Licensed Thermal Power (MWt): 3411
4. Nameplate Rating (Gross MWe): 1280 x 0.95 = 1216
5. Design Electrical Rating (Net MWe): 1130
6. Maximum Dependable Capacity (Gross MWe): 1122
7. Maximum Dependable Capacity (Net MWe): 1080
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	744.0	29,952.0
12. Hours Reactor Was Critical	725.8	725.8	17,023.8
13. Reactor Reserve Shutdown Hours	0	0	2,171.8
14. Hours Generator On-Line	721.8	721.8	16,349.6
15. Unit Reserve Shutdown Hours	0	0	1,508.7
16. Gross Thermal Energy (MWH)	2,397,367	2,397,367	51,292,977
17. Gross Electrical Energy (MWH)	778,420	778,420	16,763,835
18. Net Electrical Energy (MWH)	746,591	746,591	15,779,910
19. Unit Service Factor	97.0	97.0	54.6
20. Unit Availability Factor	97.0	97.0	59.6
21. Unit Capacity Factor (MDC Net)	92.9	92.9	48.8
22. Unit Capacity Factor (DER Net)	88.8	88.8	45.6
23. Unit Forced Outage Rate	3.0	3.0	20.9

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUELING, APRIL 9, 1980; 83 DAYS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type^1	Duration (Hours)	Reason^2	Method of Shutting Down Reactor^3	Licensee Event Report No.	System Code^4	Component Code^5	Cause & Corrective Action to Prevent Recurrence
80-01	01/04/80	F	0	H	5				TURBINE LOAD WAS REDUCED IN RESPONSE TO HIGH CONDENSER BACKPRESSURE CONDITION.
80-02	01/06/80	F	22.2	A	1				THE OUTAGE WAS TAKEN DUE TO DETERIORATING VACUUM TO REPAIR A CONDENSER VACUUM LEAK DUE TO A BROKEN TURBINE GENERATOR MAIN BEARING PEDESTAL OIL DRAIN LINE. THE LINE WAS CAPPED AND THE PEDESTAL DRAIN HOLE WAS PLUGGED.

SUMMARY

OPERATION WAS AT NEAR FULL POWER DURING THE REPORT PERIOD. LOAD REDUCTIONS OCCURRED DUE TO HIGH CONDENSER BACK-PRESSURE PROBLEMS UNTIL A BROKEN DRAIN LINE WAS DISCOVERED UNDER THE TURBINE-GENERATOR MAIN BEARING OIL PEDESTAL DRAIN. THIS WAS SUBSEQUENTLY CAPPED AND THE HOLE PLUGGED.

LEGEND

^{1}Type:
F Forced
S Scheduled

^{2}Reason:
A Equipment Failure
B Maintenance or Test
C Refueling
D Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

^{3}Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

^{4}Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

^{5}Exhibit H:
Same source.

TURKEY POINT 3

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Florida/Dade 25 miles S of Miami
2.	Electric Reliability Council	Southeastern
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2200
5.	Design Electrical Rating (MWE Net)	693
6.	Date of Initial Criticality	October 20, 1972
7.	Date of Commercial Operation	December 14, 1972
8.	Condenser Cooling Method	Recirculating Canal System
9.	Condenser Cooling Water Source	Biscayne Bay

Utility & Contractor Information

10.	Licensee	Florida Power and Light Company
11.	Corporate Address	9250 West Flagler Street P. O. Box 013100 Miami, Florida 33101
12.	Corporate Contact	R. E. Uhrig, V.P. Nuclear Affairs
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. Lewis
18.	IE Senior Inspector	E. H. Verdery
19.	IE Principal Inspector	R.J. Vogt-Lowell
20.	Licensing Project Manager	M. Grotenhuit
21.	Docket Number	50-250
22.	License Number/Date of Issue	DPR-31, 7/19/72
23.	Public Document Room	Environmental and Urban Affairs Library Florida International University Miami, Florida 33199

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO PART 21 REQUIREMENTS, FPL CORPORATE QA MANUAL QA PROCEDURE, QP 16.4, PARAGRAPH 5.2 AND FIGURE 16.4-1 DOES NOT REQUIRE THAT ALL DEVIATIONS BE FORMALLY EVALUATED AND DOCUMENTED AS PART 21 EVALUATIONS. SINCE FORMAL PART 21 EVALUATIONS ARE NOT PERFORMED AND DOCUMENTED IN ALL CASES, IT FOLLOWS THAT THE INFORMING OF THE RESPONSIBLE OFFICER (21.21(A)(2)), THE WRITTEN REPORTS TO THE COMMISSION CONTAIN THE REQUIRED INFORMATION (21.21(B)(3)), AND THE REQUIRED RECORDS BE MAINTAINED (21.51) REQUIREMENTS CANNOT BE MET (7935 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

+ RETURNED TO SERVICE 2/8/80 FOLLOWING EXTENDED REFUELING OUTAGE THAT STARTED 12/1/79.

+ LAST IE SITE INSPECTION DATE: FEBRUARY 6, 1980

+ INSPECTION REPORT NO: 50-250/80-06

SUMMARY

INSPECTION SUMMARY

+ INSPECTION NOVEMBER 14-15 (79-32): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 7 INSPECTOR-HOURS ON SITE IN THE AREA OF IE BULLETIN 79-14, "SEISMIC ANALYSIS FOR AS-BUILT SAFETY RELATED PIPING SYSTEMS." NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION DECEMBER 10-14 (79-34): THIS SPECIAL, ANNOUNCED INSPECTION INVOLVED 28 INSPECTOR-HOURS ON SITE IN THE AREA OF PREVIOUS INSPECTION FINDINGS RELATING TO THE QUALITY ASSURANCE INSPECTION CONDUCTED ON MARCH 12-16 AND 19-21, 1979. THE FINDINGS OF THIS FOLLOWUP INSPECTION SHOW EVIDENCE THAT FP&L MANAGEMENT HAS INCREASED THEIR PARTICIPATION IN QUALITY ASSURANCE ACTIVITIES. MANAGEMENT SUPPORT OF QUALITY ASSURANCE ACTIVITIES AND PROCEDURE COMPLIANCE WAS EVIDENCED BY GENERAL OFFICE AND PLANT MANAGEMENT CORRESPONDENCE TO APPROPRIATE PERSONNEL. ACCEPTANCE OF THE INCREASED MANAGEMENT INVOLVEMENT WAS EVIDENCED BY PERSONNEL CONCERN REGARDING PROPER PROCEDURE IMPLEMENTATION AND ATTITUDE CHANGE OF QUALITY ASSURANCE, ENGINEERING DEPARTMENT, AND FIELD PERSONNEL.

INSPECTION NOVEMBER 1-30 (79-36): THIS ROUTINE INSPECTION INVOLVED 52 INSPECTOR-HOURS ON SITE IN THE AREAS OF PREPARATION FOR UNIT 3 REFUELING, PLANT PHYSICAL PROTECTION, SAFETY ASPECTS OF INADVERTENT SAFETY SYSTEM ACTUATIONS, PLANT OPERATIONS, FOLLOWUP ON REPORTABLE EVENTS. OF THE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION DECEMBER 11-14 (79-37): INCLUDED REVIEW OF THE LICENSEE'S RESPONSE TO IE BULLETIN 79-16; SECURITY PLAN; SECURITY ORGANIZATION; PHYSICAL BARRIERS (PROTECTED AREA); PHYSICAL BARRIERS (VITAL AREA); ACCES CONTROL (VITAL AREA); DETECTION AIDS; AND COMPENSATORY MEASURES. THE INSPECTION INVOLVED 13 INSPECTOR-HOURS ON SITE BY 1 NRC INSPECTOR. THE INSPECTION WAS BEGUN DURING AN OFFSHIFT PERIOD; 2 1/2 INSPECTION HOURS WERE ACCOMPLISHED DURING OFFSHIFT PERIODS. THE LICENSEEE WAS FOUND TO BE IN COMPLIANCE WITH NRC REQUIREMENTS WITHIN THE EIGHT AREAS EXAMINED DURING THE INSPECTION.

INSPECTION DECEMBER 12-14 (79-38): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 10 INSPECTOR-HOURS ON SITE IN THE AREAS OF PIPE SUPPORT BASEPLATE DESIGNS USING CONCRETE EXPANSION ANCHOR BOLTS-PROCEDURES, WORK ACTIVITIES AND RECORDS; SEISMIC ANALYSIS FOR AS-BUILT SAFETY-RELATED PIPING-PROCEDURES, WORK ACTIVITIES AND RECORDS. OF THE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION DECEMBER 19-21 (79-39): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 14 INSPECTOR-HOURS ON SITE IN THE AREAS OF ADMINISTRATIVE CONTROLS INCLUDING ENVIRONMENTAL PROCEDURES AND ANNUAL AUDIT OF APPENDIX B TECHNICAL SPECIFICATIONS IMPLEMENTATION; REVIEW OF MONITORING, SURVEILLANCE AND SPECIAL STUDY PROGRAMS; REVIEW OF CHEMICAL AND THERMAL MONITORING OF PLANT COOLING WATER; REVIEW OF PLANT REPORTING REQUIREMENTS; REVIEW OF PREVIOUS INSPECTION FINDINGS; REVIEW OF LICENSEE EVENT REPORTS (LERS). OF THE SIX AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN FIVE AREAS; ONE APPARENT ITEM OF NONCOMPLIANCE WAS FOUND IN ONE AREA (DEFICIENCY - APPROVAL OF PROCEDURES).

INSPECTION DECEMBER 1-20 (79-41): THIS ROUTINE INSPECTION INVOLVED 81 INSPECTOR-HOURS ON SITE IN THE AREAS OF RECEIPT OF NEW FUEL, REFUELING ACTIVITIES, PLANT OPERATIONS, MAINTENANCE ACTIVITIES, DEBRIS IN UNIT 3 STEAM GENERATORS, RADIATION PROTECTION-OPERATIONS. OF THE SIX AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN FIVE AREAS; ONE APPARENT ITEM OF NONCOMPLIANCE WAS FOUND IN ONE AREA (INFRACTION - FAILURE TO FOLLOW PROCEDURE).

INSPECTION DECEMBER 26-27 (79-42): NUCLEAR MATERIAL CONTROL AND ACCOUNTING PROCEDURES REVIEW; AUDIT OF RECORDS AND REPORTS; AND STORAGE AND INTERNAL CONTROL. THE INSPECTION INVOLVED 7 INSPECTOR-HOURS ON SITE BY ONE NRC INSPECTOR AND WAS BEGUN DURING THE REGULAR HOURS. THE LICENSEE WAS FOUND TO BE IN COMPLIANCE WITH NRC REQUIREMENTS IN THE THREE AREAS EXAMINED DURING THE INSPECTION.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

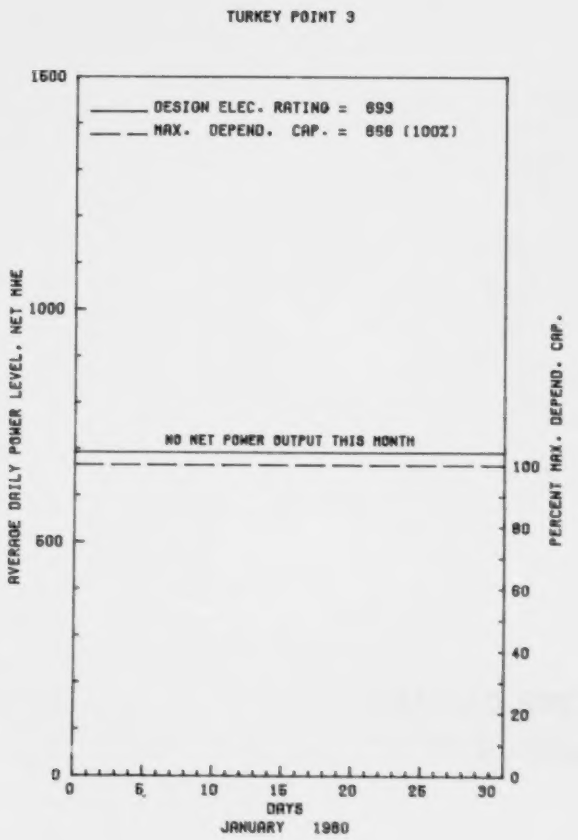
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-030/ 03L-0	10/04/79	11/05/79	30 DAY LER	RADIATION MONITORS R11 AND R12 OUT OF SERVICE FOR 2½ HOURS.
79-036/ 03L/0	11/28/79	12/28/79	30 DAY LER	AUXILIARY FEED PUMP A FAILED TO START WHEN TESTED. CAUSE UNKNOWN.
79-037/ 03L-0	12/01/79	12/31/79	30 DAY LER	OPERATOR FAILED TO OPEN SAMPLE ISOLATION VALVES FOR STEAM GENERATORS THUS ISOLATING A RADIATION MONITOR.
79-038/ 03L-0	12/01/79	12/31/79	30 DAY LER	MAIN STEAM ISOLATION VALVE FAILED TO FULLY CLOSE.
79-039/ 03L-0	12/03/79	12/31/79	30 DAY LER	FOREIGN MATERIAL FOUND IN 3B AND 3C STEAM GENERATORS PRIMARY SIDE HOT LEG AREAS.

REVIEWED BY OMPA LOEB: T. C. Cintula DATE: 3·19·80

REVIEWED BY NRR DOR: M Grotenhuis DATE: 3-20-80

TURKEY POINT 3

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

ITEM 21 CALCULATED BY USING A WEIGHTED AVERAGE

1. Docket Number: 50-250
2. Utility Contact: V. T. Chilson (305) 552-3824
3. Licensed Thermal Power (MWt): 2200
4. Nameplate Rating (Gross MWe): 894 x 0.85 = 760
5. Design Electrical Rating (Net MWe): 693
6. Maximum Dependable Capacity (Gross MWe): 700
7. Maximum Dependable Capacity (Net MWe): 666
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	744.0	62,745.[illegible]
12. Hours Reactor Was Critical	.0	.0	47,269.4
13. Reactor Reserve Shutdown Hours	.0	.0	213.3
14. Hours Generator On-Line	.0	.0	45,690.2
15. Unit Reserve Shutdown Hours	.0	.0	121.8
16. Gross Thermal Energy (MWH)	0	0	91,468,634
17. Gross Electrical Energy (MWH)	0	0	29,081,000
18. Net Electrical Energy (MWH)	-2,287	-2,287	27,519,807
19. Unit Service Factor	.0	.0	72.8
20. Unit Availability Factor	.0	.0	73.0
*21. Unit Capacity Factor (MDC Net)	.0	.0	66.4
22. Unit Capacity Factor (DER Net)	.0	.0	63.3
23. Unit Forced Outage Rate	.0	.0	2.6

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 02/08/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
23	12/01/79	S	744.0	C	4		RC	FUELXX	UNIT NO. 3 WAS REMOVED FROM SERVICE FOR SCHEDULED REFUELING, MAINTENANCE, AND INSPECTIONS. OUTAGE WAS EXTENDED TO REPAIR FAILED MECHANICAL SEAL ON REACTOR COOLANT PUMP NO. 3C. CORRECTIVE ACTIONS INCLUDED REPLACING A SHEARED LOCK PIN AND SHAFT SLEEVE ON SHAFT SEAL ASSEMBLY NO. 1.

SUMMARY

THE UNIT REMAINED SHUTDOWN THROUGHOUT THE REPORT PERIOD FOR A REFUELING, MAINTENANCE, AND INSPECTION OUTAGE.

LEGEND

[1] Type:
F–Forced
S–Scheduled

[2] Reason:
A–Equipment Failure
B–Maintenance or Test
C–Refueling
D–Regulatory Restriction
E–Operator Training & License Examination
F–Administrative
G–Operational Error
H–Other

[3] Method:
1–Manual
2–Manual Scram
3–Automatic Scram
4–Continuations
5–Load Reduction
9–Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

TURKEY POINT 4

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Florida/Dade 25 miles S of Miami
2.	Electric Reliability Council	Southeastern
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2200
5.	Design Electrical Rating (MWE Net)	693
6.	Date of Initial Criticality	June 11, 1973
7.	Date of Commercial Operation	September 7, 1973
8.	Condenser Cooling Method	Recirculating Canal System
9.	Condenser Cooling Water Source	Biscayne Bay

Utility & Contractor Information

10.	Licensee	Florida Power and Light Company
11.	Corporate Address	9250 West Flagler Street P. O. Box 013100 Miami, Florida 33101
12.	Corporate Contact	R. E. Uhrig, V. P. Nuclear Affairs
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. Lewis
18.	IE Senior Inspector	E.H. Verdery
19.	IE Principal Inspector	R.J. Vogt-Lowell
20.	Licensing Project Manager	M. Grotenhuis
21.	Docket Number	50-251
22.	License Number/Date of Issue	DPR-41, 4/10/73
23.	Public Document Room	Environmental and Urban Affairs Library Florida International University Miami, Florida 33199

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO PART 21 REQUIREMENTS, FPL CORPORATE QA MANUAL QA PROCEDURE, QP 16.4, PARAGRAPH 5.2 AND FIGURE 16.4-1 DOES NOT REQUIRE THAT ALL DEVIATIONS BE FORMALLY EVALUATED AND DOCUMENTED AS PART 21 EVALUATIONS. SINCE FORMAL PART 21 EVALUATIONS ARE NOT PERFORMED AND DOCUMENTED IN ALL CASES, IT FOLLOWS THAT THE INFORMING OF THE RESPONSIBLE OFFICER (21.21(A)(2)), THE WRITTEN REPORTS TO THE COMMISSION CONTAIN THE REQUIRED INFORMATION (21.21(B)(3)), AND THE REQUIRED RECORDS BE MAINTAINED (21.51) REQUIREMENTS CANNOT BE MET.
(7935 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

UNIT OPERATING AT 100% POWER.

LAST IE SITE INSPECTION DATE: FEBRUARY 6, 1980

INSPECTION REPORT NO: 50-251/80-06

SUMMARY

INSPECTION SUMMARY

INSPECTION NOVEMBER 14-15 (79-32): THIS ROUTINE ANNOUNCED INSPECTION INVOLVED 7 INSPECTOR-HOURS ON SITE IN THE AREA OF IE BULLETIN 79-14, "SEISMIC ANALYSIS FOR AS-BUILT SAFETY RELATED PIPING SYSTEMS." NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION DECEMBER 10-14 (79-34): THIS SPECIAL, ANNOUNCED INSPECTION INVOLVED 28 INSPECTOR-HOURS ON SITE IN THE AREA OF PREVIOUS INSPECTION FINDINGS RELATING TO THE QUALITY ASSURANCE INSPECTION CONDUCTED ON MARCH 12-16 AND 19-21, 1979. THE FINDINGS OF THIS FOLLOWUP INSPECTION SHOW EVIDENCE THAT FP&L MANAGEMENT HAS INCREASED THEIR PARTICIPATION IN QUALITY ASSURANCE ACTIVITIES. MANAGEMENT SUPPORT OF QUALITY ASSURANCE ACTIVITIES AND PROCEDURE COMPLIANCE WAS EVIDENCED BY GENERAL OFFICE AND PLANT MANAGEMENT CORRESPONDENCE TO APPROPRIATE PERSONNEL. ACCEPTANCE OF THE INCREASED MANAGEMENT INVOLVEMENT WAS EVIDENCED BY PERSONNEL CONCERN REGARDING PROPER PROCEDURE IMPLEMENTATION AND ATTITUDE CHANGE OF QUALITY ASSURANCE, ENGINEERING DEPARTMENT, AND FIELD PERSONNEL.

INSPECTION NOVEMBER 1-30 (79-36): THIS ROUTINE INSPECTION INVOLVED 52 INSPECTOR-HOURS ON SITE IN THE AREAS OF PLANT PHYSICAL PROTECTION, SAFETY ASPECTS OF INADVERTENT SAFETY SYSTEM ACTUATIONS, PLANT OPERATIONS, FOLLOWUP ON REPORTABLE EVENTS. OF THE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION DECEMBER 11-14 (79-37): INCLUDED REVIEW OF THE LICENSEE'S RESPONSE TO IE BULLETIN 79-16; SECURITY PLAN; SECURITY ORGANIZATION; PHYSICAL BARRIERS (PROTECTED AREA); PHYSICAL BARRIERS (VITAL AREA); ACCESS CONTROL (VITAL AREA); DETECTION AIDS; AND COMPENSATORY MEASURES. THE INSPECTION INVOLVED 13 INSPECTOR-HOURS ON SITE BY ONE NRC INSPECTOR. THE INSPECTION WAS BEGUN DURING AN OFFSHIFT PERIOD; 2 1/2 INSPECTION HOURS WERE ACCOMPLISHED DURING OFFSHIFT PERIODS. THE LICENSEE WAS FOUND TO BE IN COMPLIANCE WITH NRC REQUIREMENTS WITHIN THE EIGHT AREAS EXAMINED DURING THE INSPECTION.

INSPECTION DECEMBER 12-14 (79-38): THIS ROUTINE UNANNOUNCED INSPECTION INVOLVED 9 INSPECTOR-HOURS ON SITE IN THE AREAS OF PIPE SUPPORT BASEPLATE DESIGNS USING CONCRETE EXPANSION ANCHOR BOLTS-PROCEDURES, WORK ACTIVITIES AND RECORDS; SEISMIC ANALYSIS FOR AS-BUILT SAFETY-RELATED PIPING-PROCEDURES, WORK ACTIVITIES AND RECORDS. OF THE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION DECEMBER 19-21 (79-39): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 14 INSPECTOR-HOURS ON SITE IN THE AREAS OF ADMINISTRATIVE CONTROLS INCLUDING ENVIRONMENTAL PROCEDURES AND ANNUAL AUDIT OF APPENDIX B TECHNICAL SPECIFICATIONS IMPLEMENTATION; REVIEW OF MONITORING, SURVEILLANCE AND SPECIAL STUDY PROGRAMS; REVIEW OF CHEMICAL AND THERMAL MONITORING OF PLANT COOLING WATER; REVIEW OF PLANT REPORTING REQUIREMENTS; REVIEW OF PREVIOUS INSPECTION FINDINGS; REVIEW OF LICENSEE EVENT REPORTS (LERS). OF THE SIX AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN FIVE AREAS; ONE APPARENT ITEM OF NONCOMPLIANCE WAS FOUND IN ONE AREA (DEFICIENCY - APPROVAL OF PROCEDURES).

INSPECTION DECEMBER 1-20 (79-41): THIS ROUTINE INSPECTION INVOLVED 37 INSPECTOR-HOURS ON SITE IN THE AREAS OF PLANT OPERATIONS, MAINTENANCE ACTIVITIES, UNIT 4 REACTOR TRIPS, RADIATION PROTECTION-OPERATIONS. OF THE FOUR AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN THREE AREAS; ONE APPARENT ITEM OF NONCOMPLIANCE WAS FOUND IN ONE AREA (INFRACTION - FAILURE TO FOLLOW PROCEDURES).

INSPECTION DECEMBER 26-27 (79-42): NUCLEAR MATERIAL CONTROL AND ACCOUNTING PROCEDURES REVIEW; AUDIT OF RECORDS AND REPORTS; AND STORAGE AND INTERNAL CONTROL. THE INSPECTION INVOLVED 7 INSPECTOR-HOURS ON SITE BY ONE NRC INSPECTOR AND WAS BEGUN DURING THE REGULAR HOURS. THE LICENSEE WAS FOUND TO BE IN COMPLIANCE WITH NRC REQUIREMENTS IN THE THREE AREAS EXAMINED DURING THE INSPECTION.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
		NONE		

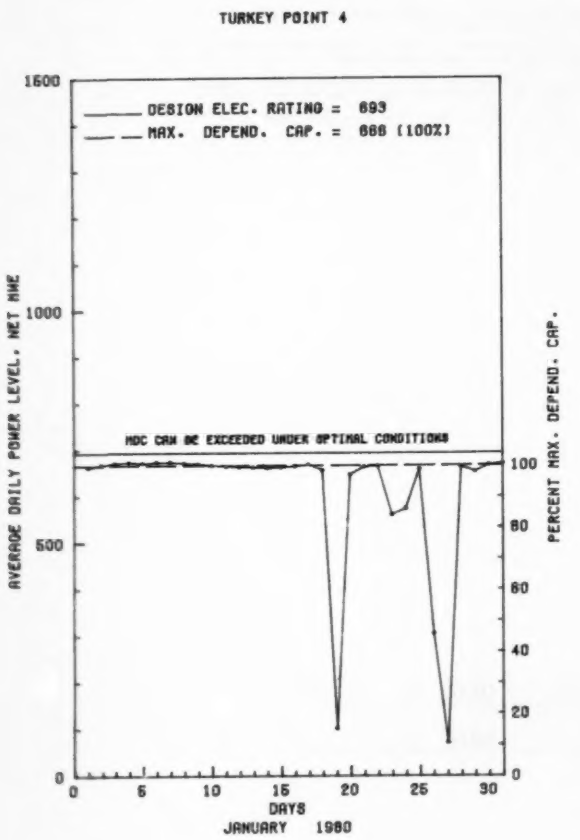
REVIEWED BY OMPA LOEB: T.C. Cintula **DATE:** 3•19•80

REVIEWED BY NRR DOR: M. Grotenhuis **DATE:** 3-20-80

GPO 878-667

TURKEY POINT 4

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

ITEM 21 CALCULATED USING A WEIGHTED AVERAGE

1. Docket Number: 50-25:
2. Utility Contact: V. T. Chilson (305) 552-3824
3. Licensed Thermal Power (MWt): 2200
4. Nameplate Rating (Gross MWe): 894 x 0.85 = 760
5. Design Electrical Rating (Net MWe): 693
6. Maximum Dependable Capacity (Gross MWe): 700
7. Maximum Dependable Capacity (Net MWe): 666
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	744.0	56,473.0
12. Hours Reactor Was Critical	730.1	730.1	41,557.9
13. Reactor Reserve Shutdown Hours	12.3	12.3	166.6
14. Hours Generator On-Line	696.3	696.3	39,878.8
15. Unit Reserve Shutdown Hours	12.3	12.3	31.2
16. Gross Thermal Energy (MWH)	1,439,848	1,499,848	82,718,828
17. Gross Electrical Energy (MWH)	477,570	477,570	26,381,048
18. Net Electrical Energy (MWH)	453,444	453,444	24,933,440
19. Unit Service Factor	93.6	93.6	70.6
20. Unit Availability Factor	95.2	95.2	70.7
*21. Unit Capacity Factor (MDC Net)	91.5	91.5	66.8
22. Unit Capacity Factor (DER Net)	87.9	87.9	63.8
23. Unit Forced Outage Rate	.4	.4	3.1

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): SCHED. MAINT. & INSPECT. - APRIL 27 - JUNE 10, 1980.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type¹	Duration (Hours)	Reason²	Method of Shutting Down Reactor³	Licensee Event Report No.	System Code⁴	Component Code⁵	Cause & Corrective Action to Prevent Recurrence
1	01/19/80	S	15.4	B	1		ZZ	ZZZZZZ	UNIT NO. 4 WAS REMOVED FROM SERVICE TO PERFORM INTEGRATED SAFEGUARDS SURVEILLANCE TEST. (NUCLEAR SYSTEM).
2	01/19/80	F	2.8	A	3		HB	ZZZZZZ	UNIT WAS TRIPPED BY STEAM GENERATOR LEVEL PROTECTION SYSTEM DUE TO A TRANSIENT CONDITION WHILE INCREASING LOAD AFTER THE UNIT WAS RETURNED TO SERVICE. (NON-NUCLEAR SYSTEM).
3	01/23/80	F	.0	A	5		HC	HTEXCH	LOAD REDUCTION TO REPAIR CONDENSER TUBE LEAK. (NON-NUCLEAR SYSTEM).
4	01/26/80	S	29.4	A	1		HA	MECFUN	UNIT WAS REMOVED FROM SERVICE TO REPAIR TURBINE CONTROL OIL SYSTEM. CORRECTIVE ACTIONS INCLUDED REPLACING TURBINE GOVERNOR IMPELLER SEAL SLEEVE. (NON-NUCLEAR SYSTEM).

SUMMARY

OPERATION WAS AT NEAR FULL POWER DURING THE REPORT PERIOD, WITH OUTAGES ON 01/19/80 FOR THE INTEGRATED SAFEGUARDS SURVEILLANCE TEST, AND ON 01/26/80 TO REPAIR THE TURBINE CONTROL OIL SYSTEM. ON 01/19/80 THE UNIT WAS ALSO TRIPPED BY THE STEAM GENERATOR LEVEL PROTECTION SYSTEM DUE TO A TRANSIENT CONDITION. LOAD WAS REDUCED ON 01/23/80 TO REPAIR A CONDENSER TUBE LEAK.

LEGEND

¹Type:
- F--Forced
- S--Scheduled

²Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

³Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

⁴Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

⁵Exhibit H: Same source.

VERMONT YANKEE

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Vermont/Windham 5 miles S of Brattleboro
2.	Electric Reliability Council	NPCC
3.	Type of Reactor	BWR
4.	Power Level (MWT)	1593
5.	Design Electrical Rating (MWE Net)	514
6.	Date of Initial Criticality	March 24, 1972
7.	Date of Commercial Operation	November 30, 1972
8.	Condenser Cooling Method	Closed Cycle Cooling Towers
9.	Condenser Cooling Water Source	Connecticut River

Utility & Contractor Information

10.	Licensee	Vermont Yankee Nuclear Power Corp.
11.	Corporate Address	20 Turnpike Road Westboro, Massachusetts 01581
12.	Corporate Contact	D. W. Edwards Assistant to the Vice President
13.	Architect/Engineer	Ebasco
14.	Nuclear Steam Supply System	GE
15.	Constructor	Ebasco

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	IE Senior Inspector	E. McCabe
19.	IE Principal Inspector	T. F. Stetka
20.	Licensing Project Manager	V. L. Rooney
21.	Docket Number	50-271
22.	License Number/Date of Issue	DPR-28, 3/21/72
23.	Public Document Room	Brooks Memorial Library 224 Main Street Brattleboro, Vermont 06301

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

- None

FACILITY ITEMS (PLANS AND PROCEDURES):

- The plant was placed in cold shutdown on 1/31/80 in conformance with the NRC's Show Cause Order dated 1/2/80. Modifications were completed to achieve full compliance with NUREG 0578 (Category A) items. Automatic Depressurization System air supply check valves were replaced and leak tested as followup actions associated with IE Bulletin 80-01. The plant returned to power operation on 2/4/80.

MANAGERIAL ITEMS:

None

PLANT STATUS:

- Shutdown on 01/31/80 for modifications.

- LAST IE SITE INSPECTION DATE: 2/8/80
- INSPECTION REPORT NO: 50-271/80-02

SUMMARY

INSPECTION SUMMARY

- 50-271/79-20 - Nov 19-21: Routine, unannounced inspection by a region based inspector (21 hours) of pipe support and restraint system surveillance and the Inservice Inspection Program for Class 1 and 2 restraints. Noncompliances: None.
- 50-271/80-01 - Jan 10: Management meeting held at NRC request to discuss details related to the establishment of an onsite office for the purpose of placing two full time NRC inspectors at the facility. All parties concurred on administrative and operational considerations needed to establish an onsite NRC resident office.

REPORTS RECEIVED FROM LICENSEE

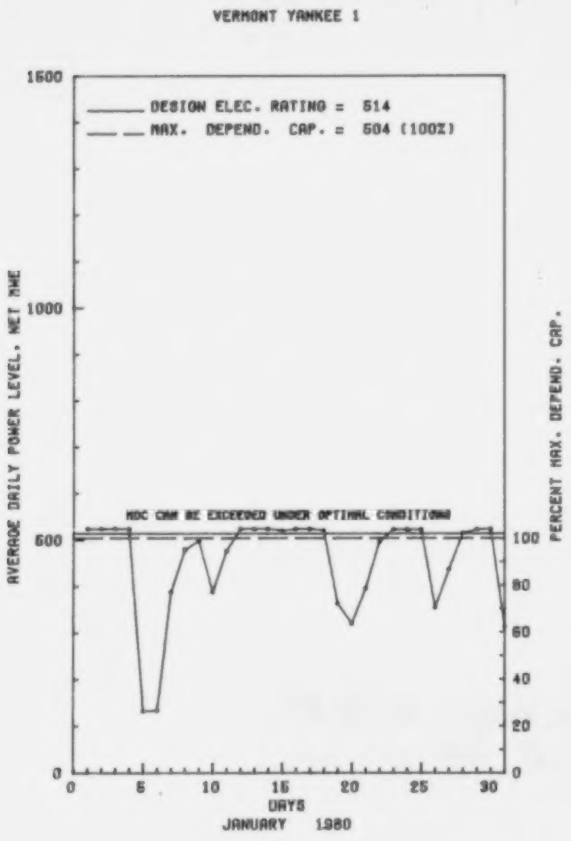
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-032/03L	12/04/79	01/02/80	30 Day LER	Hanger CST-HD 15B found inoperable during routine inspection
79-033/01T	12/05/79	12/19/79	14 Day LER	Pipe supports in five plant systems found not to meet seismic design requirements
79-034/01T	12/11/79	12/19/79	14 Day LER	Piping supports on the service water (1) and RHR (3) found inadequate for seismic design loads
79-035/03L	12/15/79	01/14/80	30 Day LER	MSIV V2-80A would not fully open during testing due to actuator spring plate binding on spring guide rods
79-036/03L	12/19/79	01/15/80	30 Day LER	DPIS 13-84, RCIC hi stm line dp switch setpoint drift

REVIEWED BY OMPA LOEB: Joseph J. Rizzo **DATE:** 3-19-80

REVIEWED BY NRR DOR: V. Rooney **DATE:** 3/20/80

VERMONT YANKEE

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-271
2. Utility Contact: R. M. Sjogren (617) 366-9011 X2281
3. Licensed Thermal Power (MWt): 1593
4. Nameplate Rating (Gross MWe): 626 x 0.9 = 563
5. Design Electrical Rating (Net MWe): 514
6. Maximum Dependable Capacity (Gross MWe): 535
7. Maximum Dependable Capacity (Net MWe): 504
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	744.0	64,538.8
12. Hours Reactor Was Critical	723.0	723.0	51,881.6
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	710.1	710.1	50,037.9
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,031,397	1,031,397	70,256,008
17. Gross Electrical Energy (MWH)	351,821	351,821	23,321,491
18. Net Electrical Energy (MWH)	337,540	337,540	22,105,905
19. Unit Service Factor	95.4	95.4	77.5
20. Unit Availability Factor	95.4	95.4	77.5
21. Unit Capacity Factor (MDC Net)	90.0	90.0	68.0
22. Unit Capacity Factor (DER Net)	88.3	88.3	66.6
23. Unit Forced Outage Rate	3.7	3.7	6.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 02/04/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-1	01/05/80	F	27.0	A	1		CF	VALVEX	EXCESSIVE DRYWELL LEAKAGE RATE CAUSED BY FAILURE OF RHR81B PACKING. VALVE WAS REPACKED.
80-02	01/10/80	S	.0	H	5		RA	CONROD	CONTROL ROD PATTERN ADJUSTMENT.
80-03	01/18/80	S	.0	H	5		RA	CONROD	TURBINE SURVEILLANCE AND CONTROL ROD PATTERN EXCHANGE.
80-04	01/20/80	F	.0	H	5		ZZ	ZZZZZZ	INDICATION OF EXCESSIVE VIBRATION OF TURBINE GENERATOR BEARINGS DUE TO LOOSE BOLTING ON EXCITER CASE FLANGE. TIGHTENED SAME.
80-05	01/26/80	S	.0	H	5		RA	CONROD	CONTROL ROD PATTERN EXCHANGE
80-06	01/31/80	S	6.9	D	9		ZZ	ZZZZZZ	SHUTDOWN FOR MODIFICATIONS REQUIRED BY NUREG 0578.

SUMMARY

OPERATION WAS AT NEAR FULL POWER DURING THE REPORT PERIOD, WITH OUTAGES ON 01/05/80 FOR VALVE REPACKING AND ON 01/31/80 FOR MODIFICATIONS REQUIRED BY NUREG-0578. LOAD REDUCTIONS OCCURRED FOR CONTROL ROD ADJUSTMENT AND TO TIGHTEN BOLTINGS CAUSING EXCESSIVE VIBRATION OF TURBINE GENERATOR BEARINGS.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

YANKEE ROWE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Massachusetts/Franklin 25 miles NE of Pittsfield
2.	Electric Reliability Council	NPCC
3.	Type of Reactor	PWR
4.	Power Level (MWT)	600
5.	Design Electrical Rating (MWE Net)	175
6.	Date of Initial Criticality	August 19, 1960
7.	Date of Commercial Operation	July 1961
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Deerfield River

Utility & Contractor Information

10.	Licensee	Yankee Atomic Electric Company
11.	Corporate Address	20 Turnpike Road Westboro, Massachusetts 01581
12.	Corporate Contact	D. W. Edwards, Asst. to V.P.
13.	Architect/Engineer	Stone & Webster
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	
17.	IE Branch Chief	E. J. Brunner
18.	IE Senior Inspector	R. R. Keimig
19.	IE Principal Inspector	W. J. Lazarus
20.	Licensing Project Manager	A. Burger
21.	Docket Number	50-29
22.	License Number/Date of Issue	DPR-3, 12/24/63
23.	Public Document Room	Greenfield Community College 1 College Drive Greenfield, Massachusetts 01301

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

+ Unit shutdown on January 19, 1980 to install "Short Term Lessons Learned" items. During startup the plant experienced a MCP Flange Leak. Plant cooled down and repaired leak. The plant resumed startup and on 2/14/80 experienced a LP turbine failure.

FACILITY ITEMS (PLANS AND PROCEDURES):

+ Results of measurements, nondestructive testing and availability of parts for the LP turbine will determine the duration of the shutdown.

MANAGERIAL ITEMS:

None

PLANT STATUS:

+ Shutdown for modifications.

+ LAST IE SITE INSPECTION DATE: 2/5/80

+ INSPECTION REPORT NO: 50-29/80-02

SUMMARY

INSPECTION SUMMARY

+ 50-29/79-16 - Dec 17-19: Routine unannounced inspection of emergency planning including: licensee coordination with offsite agencies; emergency facilities, equipment, instrumentation and supplies specified in the Emergency Plan and Implementing Procedures; licensee records relating to emergency drills; licensee's management controls in the area of emergency planning; Emergency Plan Implementing Procedures; and observation of a licensee conducted emergency drill. The inspection involved 18 hours onsite by one region-based inspector. No items of noncompliance were identified.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

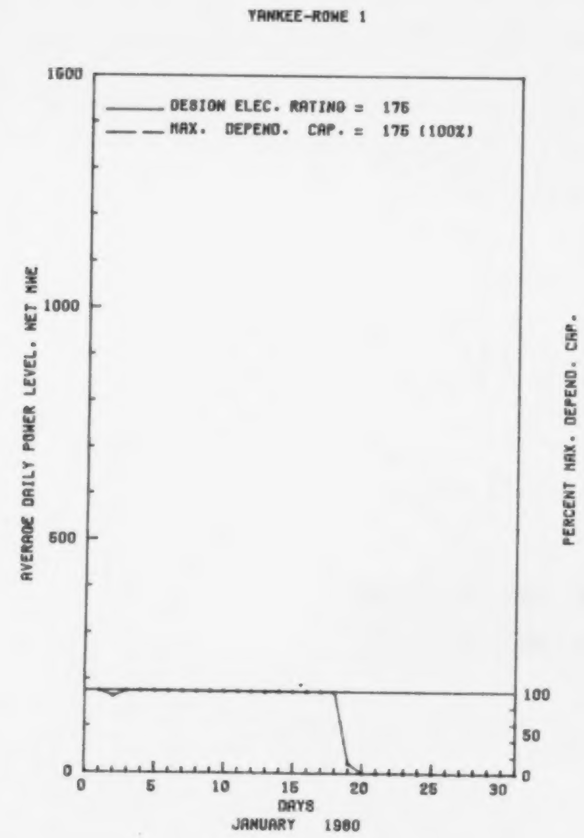
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-031/ 03L	11/30/79	12/28/79	30 Day LER	Steam generator blowdown rad monitor isolated
79-032/ 03L	12/11/79	01/10/80	30 Day LER	Primary vent stack particulate gas monitor failed

REVIEWED BY OMPA LOEB: T. C. Cintula **DATE:** 3·19·80

REVIEWED BY NRR DOR: S. J. Nowicki **DATE:** 3-21-80

YANKEE ROWE

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-029
2. Utility Contact: R. M. Sjogren (617) 366-9011 X2281
3. Licensed Thermal Power (MWt): 600
4. Nameplate Rating (Gross MWe): 185 x 1.0 = 185
5. Design Electrical Rating (Net MWe): 175
6. Maximum Dependable Capacity (Gross MWe): 180
7. Maximum Dependable Capacity (Net MWe): 175
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	744.0	168,381.0
12. Hours Reactor Was Critical	439.7	439.7	138,804.3
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	439.7	439.7	134,467.1
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	253,769	253,769	72,002,091
17. Gross Electrical Energy (MWH)	80,635	80,635	22,152,788
18. Net Electrical Energy (MWH)	75,749	75,749	20,741,893
19. Unit Service Factor	59.1	59.1	79.9
20. Unit Availability Factor	59.1	59.1	79.9
21. Unit Capacity Factor (MDC Net)	58.2	58.2	72.7
22. Unit Capacity Factor (DER Net)	58.2	58.2	72.7
23. Unit Forced Outage Rate	.0	.0	1.6

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 02/16/79

CAPACITY FACTORS
CALCULATED WITH A
WEIGHTED AVERAGE

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
1	01/19/80	S	304.3	D	1				PLANT SHUTDOWN TO INSTALL VOLTAGE REGULATORS AND MAKE TMI RELATED CHANGES.

SUMMARY

OPERATION WAS ROUTINE UNTIL 01/19/80, WHEN THE UNIT WAS SHUTDOWN TO INSTALL VOLTAGE REGULATORS AND TO MAKE TMI RELATED MODIFICATIONS.

LEGEND

[1] Type:
- F–Forced
- S–Scheduled

[2] Reason:
- A–Equipment Failure
- B–Maintenance or Test
- C–Refueling
- D–Regulatory Restriction
- E–Operator Training & License Examination
- F–Administrative
- G–Operational Error
- H–Other

[3] Method:
- 1–Manual
- 2–Manual Scram
- 3–Automatic Scram
- 4–Continuations
- 5–Load Reduction
- 9–Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

ZION 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1. Location (state/county) (nearest population center)	Illinois/Lake 40 miles N of Chicago
2. Electric Reliability Council	Main
3. Type of Reactor	PWR
4. Power Level (MWT)	3250
5. Design Electrical Rating (MWE Net)	1040
6. Date of Initial Criticality	June 19, 1973
7. Date of Commercial Operation	December 31, 1973
8. Condenser Cooling Method	Straight Through
9. Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10. Licensee	Commonwealth Edison Co.
11. Corporate Address	P. O. Box 767 Chicago, Illinois
12. Corporate Contact	D. L. Peoples
13. Architect/Engineer	Sargent & Lundy
14. Nuclear Steam Supply System	Westinghouse
15. Constructor	Various

Regulatory Information

16. IE Region responsible	III
17. IE Branch Chief	R. Heishman
18. IE Senior Inspector	R. Spessard
19. IE Resident Inspector	J.E. Kohler
20. Licensing Project Manager	E. Reeves
21. Docket Number	50-295
22. License Number/Date of Issue	DPR-39 issued 4/6/73
23. Public Document Room	Zion-Benton Public Library 2600 Emmens Avenue Zion, Illinois 60099

INSPECTION STATUS

ENFORCEMENT STATUS

SECTION 4.16, TABLE 4.16-1, ITEM 3, OF APPENDIX A TECHNICAL SPECIFICATIONS REQUIRES THAT FISH SHALL BE COLLECTED SEMI-ANNUALLY AND GAMMA ISOTOPIC ANALYSIS OF THE FISH BE CONDUCTED. CONTRARY TO THE ABOVE, THE LICENSEE FAILED TO COLLECT FISH THE SECOND HALF OF 1978 AND CONDUCT THE REQUIRED ANALYSIS.
(7907 INFRACTION)

THE OBJECTIVE OF SECTION 4.16 OF APPENDIX A TECHNICAL SPECIFICATIONS REQUIRES THE LICENSEE TO ESTABLISH A RADIOLOGICAL MONITORING PROGRAM ADEQUATE TO MEASURE CHANGES IN THE LEVELS OF ENVIRONMENTAL RADIOACTIVITY DUE TO PLANT EFFLUENTS. CONTRARY TO THE ABOVE, THE LICENSEE FAILED TO PLACE THE THERMOLUMINESCENT DOSIMETERS (TLD) AT ONSITE STATIONS NOS. 1 AND 3 ADEQUATELY IN THAT THE MONITORING OF THE TLD MATERIAL PRECLUDED MEASUREMENT OF XE-133 AND OTHER PLANT EFFLUENTS.
(7907 DEVIATION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

+ Unit 1 feedwater nozzle and pipe repair continues. Unit 1 turbine motor removed due to cracks completed. Unit 1 Bergen-Paterson steam generator snubbers removed and sent to Wyle Laboratories for testing following removal of fluid leaks and gauled insides. Unit 1 remained in cold shutdown the entire reporting period. (79-27)

PLANT STATUS

+ The unit was shut down on October 6, 1979 to begin a scheduled 8-week refueling/maintenance outage. The outage schedule has been extended by about 9 weeks due to the system and component problems discussed above. The tentative startup date is February 4, 1980.

+LAST IE SITE INSPECTION DATE: November 6 through December 21, 1979

+ INSPECTION REPORT NO: 79-27

SUMMARY

INSPECTION SUMMARY

+ Inspection on September 26-27; October 3-4, 22-23 and November 7-9, (79-21): Follow up to IE Bulletin No. 79-02, pipe support base plate designs using concrete expansion ancher bolts; review of test program and procedures; observation of testing anchor bolts; and review of quality records. The inspection involved 68 inspector hours onsite by one NRC inspector. No items of noncompliance were identified in the areas inspected.

+ Inspection on November 6 through December 21, (79-27): Routine resident inspection of plant operations, maintenance radiation protection, IE Bulletins and Circulars, licensed operator requalification training, rad waste system operation, design changes and modifications, surveillance testing on Unit 1, security and non-routine events. The inspection involved 156 inspector hours onsite by one NRC inspector. Of the areas inspected, no items of noncompliance were identified.

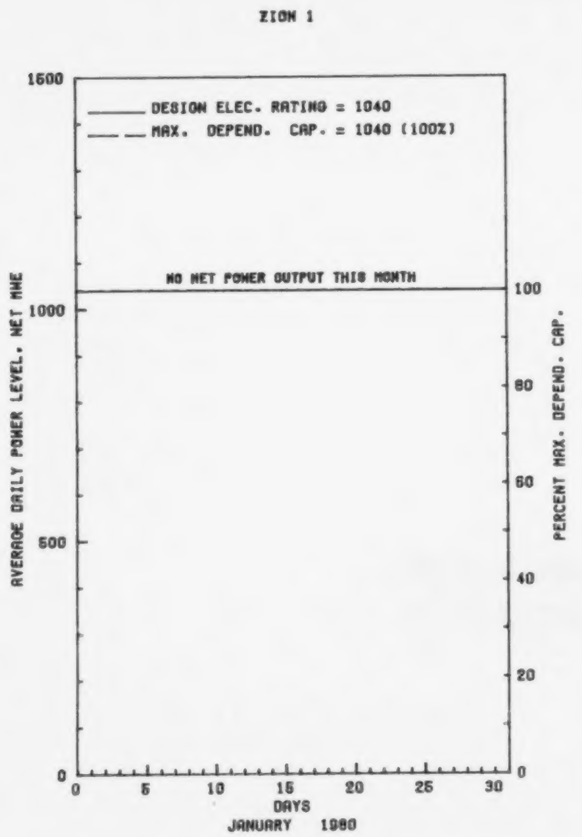
REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-92/ 03L-0	12/06/79	01/04/80	30 Day LER	Rad monitor ORT-PR02B alarmed when the indicator needle read below the alarm setpoint.
79-93/ 03L-0	12/25/79	01/24/80	30 Day LER	Radioactivity analysis of fire sump discharge sample overlooked while both units were in cold shutdown.
79-96/ 01T-0	12/28/79	01/07/80	14 Day LER	pH of effluents in condenser cooling water was not within TS.

REVIEWED BY OMPA LOEB: T.C. Cintula **DATE:** 3·19·80

REVIEWED BY NRR DOR: Ed A. Reeves **DATE:** 3-20-80

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-295
2. Utility Contact: J. COOK (312) 746-2084
3. Licensed Thermal Power (MWt): 3250
4. Nameplate Rating (Gross MWe): 1220 x 0.9 = 1098
5. Design Electrical Rating (Net MWe): 1040
6. Maximum Dependable Capacity (Gross MWe): 1085
7. Maximum Dependable Capacity (Net MWe): 1040
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	744.0	53,352.0
12. Hours Reactor Was Critical	.0	.0	37,107.7
13. Reactor Reserve Shutdown Hours	.0	.0	2,621.8
14. Hours Generator On-Line	.0	.0	36,087.3
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	0	99,383,001
17. Gross Electrical Energy (MWH)	0	0	32,232,050
18. Net Electrical Energy (MWH)	0	0	30,484,246
19. Unit Service Factor	.0	.0	67.6
20. Unit Availability Factor	.0	.0	67.6
21. Unit Capacity Factor (MDC Net)	.0	.0	54.9
22. Unit Capacity Factor (DER Net)	.0	.0	54.9
23. Unit Forced Outage Rate	100.0	100.0	14.7

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 02/12/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
1	01/01/80	F	480.0	D	4	79-077	HH	PIPEXX	UNIT DOWN FOR FEEDWATER NOZZLE REPAIR PER NRC BULLETIN #79-13; FOLLOWING REFUELING OUTAGE.
2	01/21/80	F	72.0	A	9				REPLACE 1A CHARGING PUMP.
3	01/24/80	F	192.0	A	9				REPLACE 1B RCP SEALS.

SUMMARY

THE UNIT REMAINED SHUTDOWN THROUGHOUT THE REPORT PERIOD FOR FEEDWATER NOZZLE REPAIR PER NRC BULLETIN 79-13.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

ZION 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Illinois/Lake 40 miles N of Chicago
2.	Electric Reliability Council	Main
3.	Type of Reactor	PWR
4.	Licenses Power Level (MWT)	3250
5.	Design Electrical Rating (MWE Net)	1040
6.	Date of Initial Criticality	December 24, 1973
7.	Date of Commercial Operation	September 17, 1974
8.	Condenser Cooling Method	Straight Through
9.	Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10.	Licensee	Commonwealth Edison Co.
11.	Corporate Address	P. O. Box 767 Chicago, Illinois
12.	Corporate Contact	D. L. Peoples, Manager of Nuclear Licensing
13.	Architect/Engineer	Sargent & Lundy
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Various

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Senior Inspector	R. Spessard
19.	IE Resident Inspector	J.E. Kohler
20.	Licensing Project Manager	E. Reeves
21.	Docket Number	50-304
22.	License Number/Date of Issue	DPR-48, 11/14/73
23.	Public Document Room	Zion-Benton Public Library 2600 Emmans Avenue Zion, Illinois 60099

INSPECTION STATUS

ENFORCEMENT STATUS

SECTION 4.16, TABLE 4.16-1, ITEM 3, OF APPENDIX A TECHNICAL SPECIFICATIONS REQUIRES THAT FISH SHALL BE COLLECTED SEMI-ANNUALLY AND GAMMA ISOTOPIC ANALYSIS OF THE FISH BE CONDUCTED. CONTRARY TO THE ABOVE, THE LICENSEE FAILED TO COLLECT FISH THE SECOND HALF OF 1978 AND CONDUCT THE REQUIRED ANALYSIS.
(7908 INFRACTION)

THE OBJECTIVE OF SECTION 4.16 OF APPENDIX A TECHNICAL SPECIFICATIONS REQUIRES THE LICENSEE TO ESTABLISH A RADIOLOGICAL MONITORING PROGRAM ADEQUATE TO MEASURE CHANGES IN THE LEVELS OF ENVIRONMENTAL RADIOACTIVITY DUE TO PLANT EFFLUENTS. CONTRARY TO THE ABOVE, THE LICENSEE FAILED TO PLACE THE THERMOLUMINESCENT DOSIMETERS (TLD) AT ONSITE STATIONS NOS. 1 AND 3 ADEQUATELY IN THAT THE MONITORING OF THE TLD MATERIAL PRECLUDED MEASUREMENT OF XE-133 AND OTHER PLANT EFFLUENTS.
(7908 DEVIATION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

+ Unit 2 feedwater nozzle and pipe repairs were completed.

PLANT STATUS

+ The unit started up on January 20, 1980 and is operating at full power.

+LAST IE SITE INSPECTION DATE: November 6 through December 21, 1979

+ INSPECTION REPORT NO: 79-26

SUMMARY

INSPECTION SUMMARY

+ Inspection on September 26-27; October 3-4, 22-23 and November 7-9, (79-20): Follow up to IE Bulletin No. 79-02, pipe support base plate designs using concrete expansion anchor bolts; review of test program and procedures; observation of testing anchor bolts; and review of quality records. The inspection involved 68 inspector hours onsite by one NRC inspector. No items of noncompliance were identified in the areas inspected.

+ Inspection on November 6 through December 21, (79-26): Routine resident inspection of plant operations, maintenance radiation protection, IE Bulletins and Circulars, licensed operator requalification training, rad waste system operation, design changes and modifications, surveillance testing on Unit 2, security and non-routine events. The inspection involved 156 inspector hours onsite by one NRC inspector. Of the areas inspected, no items of noncompliance were identified.

REPORTS RECEIVED FROM LICENSEE

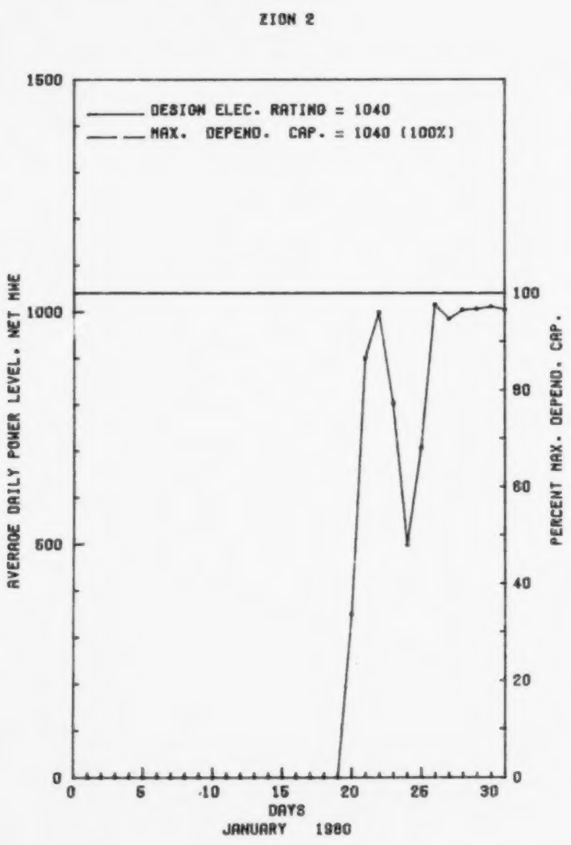
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
NONE				

REVIEWED BY OMPA LOEB: T. C. Cintula DATE: 3•19•80

REVIEWED BY NRR DOR: Ed a. Reeves DATE: 3-20-80

ZION 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-304
2. Utility Contact: J. COOK (312) 746-2084
3. Licensed Thermal Power (MWt): 3250
4. Nameplate Rating (Gross MWe): 1220 x 0.9 = 1098
5. Design Electrical Rating (Net MWe): 1040
6. Maximum Dependable Capacity (Gross MWe): 1085
7. Maximum Dependable Capacity (Net MWe): 1040
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	744.0	47,065.0
12. Hours Reactor Was Critical	284.6	284.6	33,898.2
13. Reactor Reserve Shutdown Hours	.0	.0	226.1
14. Hours Generator On-Line	268.1	268.1	33,082.0
15. Unit Reserve Shutdown Hours	.0	.0	0
16. Gross Thermal Energy (MWH)	795,372	795,372	93,430,690
17. Gross Electrical Energy (MWH)	257,770	257,770	29,978,990
18. Net Electrical Energy (MWH)	239,885	239,885	28,441,263
19. Unit Service Factor	36.0	36.0	70.3
20. Unit Availability Factor	36.0	36.0	70.3
21. Unit Capacity Factor (MDC Net)	31.0	31.0	58.1
22. Unit Capacity Factor (DER Net)	31.0	31.0	58.1
23. Unit Forced Outage Rate	64.0	64.0	18.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
REFUELING - MAY 3, 1980 - APPROX. SEVEN WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
1	10/27/79	F	456.1	D	4	79-050	HH	PIPEXX	UNIT DOWN FOR FEEDWATER NOZZLE REPAIR PER NRC BULLETIN #79-13 (CONTINUED FROM DECEMBER).
2	01/24/80	F	15.9	A	3				REACTOR TRIP CAUSED BY TRIP OF BOTH ROD DRIVE MG SETS.
3	01/24/80	F	3.9	G	3				REACTOR TRIP/TURBINE TRIP CAUSED BY A LO-LO LEVEL IN COINCIDENT WITH SF/FF MISMATCH.

SUMMARY

THE UNIT CAME ON LINE ON 01/20/80 FROM AN OUTAGE TO REPAIR FEEDWATER NOZZLES PER NRC BULLETIN 79-13. ON 01/24/80, A REACTOR TRIP WAS CAUSED BY FAILURE OF THE 2A ROD DRIVE MOTOR GENERATOR SETS, CAUSING AN OVERCURRENT CONDITION. A TURBINE /REACTOR TRIP OCCURRED THE SAME DAY FROM LOW STEAM GENERATOR LEVEL COINCIDENT WITH STEAM FLOW/FEED FLOW MISMATCH.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

SECTION 3 – APPENDIX

STATUS OF SPENT FUEL STORAGE CAPABILITY

DATA AS OF 01-31-80

Facility	Core Size [a] (No. of Assemblies)	Present Authorized Storage Pool Capacity (Fuel Assemblies)	No. of Assemblies Stored	Remaining Capacity (No. of Assemblies)	Remaining Capacity If Pending Request Approved (No. of Assemblies)	Scheduled Date For Next Refueling	Will Fill Present [b] Authorized Capacity
I. Pressurized Water Reactors							
ARKANSAS 1	177	590	176	414		12-80	1988
ARKANSAS 2 (d)	177	486	0	486		03-81	1993
BEAVER VALLEY 1	157	833	0	833		N/S	1995
CALVERT CLIFFS 1	217	1056 (c)	364 (c)	692 (c)		10-80	1984
CALVERT CLIFFS 2	217					01-81	
COOK 1	193	500	193	307	1857	N/S	1985
COOK 2	193					N/S	
CRYSTAL RIVER 3	177	256	60	196	1109	04-80	1982
DAVIS-BESSE 1	177	735	0	735		03-80	1989
FARLEY 1	157	675	0	675		N/S	1990
FORT CALHOUN 1	133	483	157	326		N/S	1985
GINNA	121	595	156	439		03-80	1987
HADDAM NECK	157	1168	340	828		05-80	1994
INDIAN POINT 1	0	288	160	128		N/S	
INDIAN POINT 2	193	482	200	282		N/S	1984
INDIAN POINT 3	193	837	64	773		N/S	1989
KEWAUNEE	121	990	120	870		05-80	1999
MAINE YANKEE	217	953	433	520		N/S	1986
MILLSTONE 2	217	667	144	523		07-80	1987
NORTH ANNA 1	157	400	0	400	966	N/S	1984
NORTH ANNA 2 (d)	157	400	0	400		N/S	
OCONEE 1	177	1224 (l)	545	(l)	679	N/S	1980
OCONEE 2	177					03-80	
OCONEE 3	177					N/S	
PALISADES	204	798	273	525		N/S	1985
POINT BEACH 1	121	1502 (c)	212 (c)	1290 (c)		N/S	1995
POINT BEACH 2	121					03-80	1995
PRAIRIE ISLAND 1	121	687 (c)	280 (c)	407 (c)		06-80	1984
PRAIRIE ISLAND 2	121					N/S	1984
RANCHO SECO 1	177	579	112	467		07-81	1987
ROBINSON 2	157	272	106	166 (g)		05-80	1980 (g)
SALEM 1	193	264	40	224	1170	09-80	1982
SAN ONOFRE 1	157	216	55	161		04-80	1983
ST LUCIE 1	217	728	60	668		03-80	1990
SURRY 1	157	1044 (c)	1983 (c)	608 (c)		06-80	1983
SURRY 2	157					N/S	
THREE MILE ISLAND 1	177	752	208	544		N/S	1989
THREE MILE ISLAND 2	177	442	0	442		N/S	1986
TROJAN	193	651	64	587		04-80	1988
TURKEY POINT 3	157	621	346 (c)	275 (c)		N/S	1987
TURKEY POINT 4	157					N/S	1987
YANKEE-ROWE 1	76	391	149	242	572	N/S	1985
ZION 1	193	868 (c)	436 (c)	432 (c)	1676	N/S	1983
ZION 2	193					05-80	1983

Facility	Core Size [a] (No. of Assemblies)	Present Authorized Storage Pool Capacity (Fuel Assemblies)	No. of Assemblies Stored	Remaining Capacity (No. of Assemblies)	Remaining Capacity If Pending Request Approved (No. of Assemblies)	Scheduled Date For Next Refueling	Will Fill Present [b] Authorized Capacity
II. Boiling Water Reactors							
BIG ROCK POINT 1	84	193	88	105	353	10-80	1983
BROWNS FERRY 1	764	3471	676	2795		N/S	1985
BROWNS FERRY 2	764	1080	400	680		09-80	1985
BROWNS FERRY 3	764	1528	354	1174		N/S	1985
BRUNSWICK 1	560	(f)	154(PWR)+320(BWR)			03-80	1986
BRUNSWICK 2	560	(f)	91(PWR)+132(BWR)			04-80	1986
COOPER STATION	548	2366	284	2082		03-80	1995
DRESDEN 1	464	672	221	451		N/S	1983
DRESDEN 2	724	2840 (c)	1069 (c)	1771 (c)	6491 (c)	N/S	1985
DRESDEN 3	724					03-80	1984
DUANE ARNOLD	368	2050	276	1774		02-80	1998
FITZPATRICK	560	760	268	492	1752	04-80	1981
HATCH 1	560	840	260	580		N/S	1988
HATCH 2	560	1120	0	1120		N/S	
HUMBOLDT BAY	172	487	251	236		N/S	
LA CROSSE	72	134	113	21	327	02-80	1983
MILLSTONE 1	580	2184	776	1408		09-80	1991
MONTICELLO	484	2237	728	1509		02-80	1990
NINE MILE POINT 1	532	1984	660	1324	2349	N/S	1990
OYSTER CREEK 1	560	1800	620	1180		N/S	1987
PEACH BOTTOM 2	764	2816	618	2198		03-80	1990
PEACH BOTTOM 3	764	2816	712	2104		03-81	1991
PILGRIM 1	580	2320	580	1740		N/S	1990
QUAD CITIES 1	724	1460	596	864		09-80	1986
QUAD CITIES 2	724	1460	492	968		N/S	1985
VERMONT YANKEE 1	368	2000	894	1106		N/S	1988
III. Independent Spent Fuel Storage Installations (h)							
Morris Operations		750 MTU (j)	310	390 MTU (j)	1490 MTU (j)		
NFS(i)		250 MTU	170 MTU	80 MTU			

(a) At each refueling outage approximately 1/3 of a PWR core and 1/4 of a BWR core is off-loaded.
(b) Some of these dates have been adjusted by staff assumptions.
(c) This is the total for both units.
(J) Plant not in commercial operation.
(e) Some spent fuel stored at Brunswick.
(f) Authorized a total 2088 BWR and 304 PWR assemblies for both pools.
(g) Robinson 2 assemblies being shipped to Brunswick for storage.
(h) Capacity is in metric tons of uranium; 1 MTU = 2 PWR assemblies or 5 BWR assemblies.
(i) No longer accepting spent fuel.
(j) Racked for 700 MTU.
(k) Reserved
(l) This is the station total.

N/S = Not Scheduled Yet

REACTOR-YEARS OF EXPERIENCE

The total of Reactor years of experience is as the sum of all calendar days for each unit, from the date that electricity was first generated until a final shutdown date or the status date, whichever comes first, divided by 365.25 days/year.

If a date is unknown, the first day of the first month of operation is substituted. Units which have not yet generated electricity but which are licensed are listed but not included in the computation.

(INFORMATION INCLUDES BOTH LICENSED AND NON–LICENSED ELECTRICAL UNITS).

LICENSED OPERATING ELECTRICAL PRODUCING UNITS

UNIT NAME	FIRST ELEC. GENERATION	YEARS EXPERIENCE
ARKANSAS 1	08/01/74	5.50
ARKANSAS 2	12/26/78	1.10
BEAVER VALLEY 1	06/14/76	3.63
BIG ROCK POINT 1	12/08/62	17.15
BROWNS FERRY 1	10/15/73	6.30
BROWNS FERRY 2	08/28/74	5.43
BROWNS FERRY 3	09/12/76	3.39
BRUNSWICK 1	12/04/76	3.16
BRUNSWICK 2	04/29/75	4.76
CALVERT CLIFFS 1	01/03/75	5.08
CALVERT CLIFFS 2	12/07/76	3.15
COOK 1	02/10/75	4.97
COOK 2	03/22/78	1.86
COOPER STATION	05/10/74	5.73
CRYSTAL RIVER 3	01/30/77	3.00
DAVIS-BESSE 1	08/28/77	2.43
DRESDEN 1	04/15/60	19.80
DRESDEN 2	04/13/70	9.80
DRESDEN 3	07/22/71	8.53
DUANE ARNOLD	05/19/74	5.71
FARLEY 1	08/18/77	2.46
FITZPATRICK	02/01/75	5.00
FORT CALHOUN 1	08/25/73	6.44
FORT ST VRAIN	12/11/76	3.14
GINNA	12/02/69	10.17
HADDAM NECK	08/07/67	12.49
HATCH 1	11/11/74	5.22
HATCH 2	09/22/78	1.36
INDIAN POINT 2	06/26/73	6.60
INDIAN POINT 3	04/27/76	3.76
KEWAUNEE	04/08/74	5.82
LA CROSSE	04/26/68	11.77
MAINE YANKEE	11/08/72	7.23
MILLSTONE 1	11/29/70	9.17
MILLSTONE 2	11/09/75	4.23
MONTICELLO	03/05/71	8.91
NINE MILE POINT 1	11/09/69	10.23
NORTH ANNA 1	04/17/78	1.79
OCONEE 1	05/06/73	6.74
OCONEE 2	12/05/73	6.16
OCONEE 3	09/01/74	5.42
OYSTER CREEK 1	09/23/69	10.36
PALISADES	12/31/71	8.09
PEACH BOTTOM 2	02/18/74	5.95
PEACH BOTTOM 3	09/01/74	5.42
PILGRIM 1	07/19/72	7.58
POINT BEACH 1	11/06/70	9.24
POINT BEACH 2	08/02/72	7.50
PRAIRIE ISLAND 1	12/04/73	6.16
PRAIRIE ISLAND 2	12/21/74	5.11
QUAD CITIES 1	04/12/72	7.81
QUAD CITIES 2	05/23/72	7.69
RANCHO SECO 1	10/13/74	5.30
ROBINSON 2	09/26/70	9.35
SALEM 1	12/25/76	3.10

LICENSED OPERATING ELECTRICAL PRODUCING UNITS - CONT.

UNIT NAME	FIRST ELEC. GENERATION	YEARS EXPERIENCE
SAN ONOFRE 1	07/16/67	12.55
ST LUCIE 1	05/07/76	3.74
SURRY 1	07/04/72	7.58
SURRY 2	03/10/73	6.90
THREE MILE ISLAND 1	06/19/74	5.62
TROJAN	12/23/75	4.11
TURKEY POINT 3	11/02/72	7.25
TURKEY POINT 4	06/21/73	6.61
VERMONT YANKEE 1	09/20/72	7.36
YANKEE-ROWE 1	11/10/60	19.23
ZION 1	06/28/73	6.60
ZION 2	12/26/73	6.10
TOTAL		446.85 YEARS

PERMANENTLY OR INDEFINITELY SHUTDOWN UNITS

UNIT NAME	FIRST ELEC. GENERATION	SHUTDOWN OR ELECTRICAL CESSATION DATE	YEARS EXPERIENCE
BONUS	08/14/64	06/01/68	3.80
CVTR	12/18/63	01/01/67	3.04
ELK RIVER	08/24/63	02/01/68	4.44
FERMI 1	08/05/66	11/29/72	6.32
HALLAM	05/29/63	09/01/64	1.26
HUMBOLDT BAY*	04/18/63	07/02/76	13.21
INDIAN POINT 1	09/16/62	10/31/74	12.12
PATHFINDER	07/25/66	10/01/67	1.19
PEACH BOTTOM 1	01/27/67	11/01/74	7.76
PIQUA	11/04/63	01/01/66	2.16
THREE MILE ISLAND 2*	04/21/78	03/28/79	.93
TOTAL			56.23 YEARS

*NOTE: HUMBOLDT BAY AND THREE MILE ISLAND 2 HAVE BEEN CHANGED FROM THE CATEGORY OF "LICENSED OPERATING ELECTRICAL PRODUCING UNITS" TO THE CLASSIFICATION OF A "PERMANENTLY OR INDEFINITELY SHUTDOWN UNITS". FOR THE COMPUTATION OF REACTOR YEARS OF EXPERIENCE, THEIR EFFECTIVE SHUTDOWN OR ELECTRICAL CESSATION DATE IS AS LISTED. BECAUSE, IN PREVIOUS EDITIONS, THESE TWO UNITS HAD BEEN CONSIDERED TO BE OPERATING UNITS, THE TOTAL YEARS OF OPERATING EXPERIENCE WILL NOT BE CONSISTENT WITH THE VALUE PUBLISHED FOR NOVEMBER 1, 1979.

NON–POWER REACTORS IN THE U.S.

1. RESEARCH REACTORS

State	Location	Licensee	Reactor Type	Docket	License Number	Date Issued	Authorized Power Level (KW)
Alabama	1. Tuskegee	Tuskegee Institute	AGN 201 No. 102	50-406	R-122	08-30-74	.0001
Arizona	2. Tucson	U. of Arizona	TRIGA Mk. I	50-113	R-52	12-05-58	100
California	3. Los Angeles	U. of California	Argonaut	50-142	R-71	10-03-60	100
	4. Berkeley	U. of California	TRIGA Mark III	50-224	R-101	08-10-66	1000
	5. Irvine	U. of California	TRIGA Mark I	50-326	R-116	11-24-69	250
	6. Santa Barbara	U. of California	L-77	50-433	R-124	12-03-74	.01
	7. Canoga Park	Rockwell International Corporation	L-77	50-94	R-40	05-17-58	.01
	8. Hawthorne	Northrop Corporate Laboratories	TRIGA Mark F	50-187	R-90	03-04-63	1000
	9. San Luis Obispo	California State Polytechnic College	AGN 201 No. 100	50-394	R-121	05-16-73	.0001
	10. Alameda County	General Electric Co.	NTR	50-73	R-33	10-31-57	100
	11. San Diego	General Atomic Co.	TRIGA Mark I	50-89	R-38	05-03-58	250
	12. San Diego	General Atomic Co.	TRIGA Mark F	50-163	R-67	07-01-60	1500
	13. San Ramon	Aerotest Operations, Inc.	TRIGA (Industrial)	50-228	R-98	07-02-65	250
Colorado	14. Jefferson County	U.S. Geological Survey, Dept. of Interior	TRIGA Mark I	50-274	R-113	02-24-69	1000
Delaware	15. Newark	U. of Delaware	AGN-201 No. 113	50-98	R-43	07-03-58	.0001
District of Columbia	16. –	Catholic U.	AGN-201 No. 101	50-77	R-31	11-15-67	.0001
Florida	17. Gainesville	U. of Florida	Argonaut	50-83	R-56	05-21-59	100
Georgia	18. Atlanta	Georgia Institute of Technology	AGN-201 No. 104	50-276	R-111	04-19-68	.0001
	19. Atlanta	Georgia Institute of Technology	Heavy Water	50-160	R-97	12-29-64	5000
Idaho	20. Pocatello	Idaho State U.	AGN-201 No. 103	50-284	R-110	10-11-67	.0001
Illinois	21. Urbana	U. of Illinois	Advanced TRIGA	50-151	R-115	07-22-69	1500
	22. Urbana	U. of Illinois	LOPRA	50-356	R-117	12-27-71	10
	23. Zion	Westinghouse Electric Corporation	NTR	50-87	R-119	01-28-72	10
Indiana	24. West Lafayette	Purdue U.	Lockheed	50-182	R-87	08-16-62	10

RESEARCH REACTORS – CONT.

State	Location	Licensee	Reactor Type	Docket	License Number	Date Issued	Authorized Power Level (KW)
Iowa	25. Ames	Iowa State U.	UTR-10	50-116	R-59	10-16-59	10
Kansas	26. Lawrence	U. of Kansas	Lockheed	50-148	R-78	06-23-61	250
	27. Manhattan	Kansas State U.	TRIGA	50-188	R-88	10-16-62	250
Maryland	28. College Park	U. of Maryland	TRIGA	50-166	R-70	10-14-60	250
	29. Bethesda	Armed Forces Radiobiology Research Institute	TRIGA	50-170	R-84	06-26-62	1000
Massachusetts	30. Lowell	U. of Lowell	GE	50-223	R-125	12-24-74	1000
	31. Cambridge	Massachusetts Institute of Technology	Heavy Water Reflected	50-20	R-37	06-09-58	5000
	32. Worcester	Worcester Polytechnic Institute	GE	50-134	R-61	12-16-59	10
Michigan	33. Ann Arbor	U. of Michigan	Pool	50-2	R-28	09-13-57	2000
	34. Lansing	Michigan State U.	TRIGA Mark I	50-294	R-114	03-21-69	250
	35. Midland	Dow Chemical Co.	TRIGA Mark I	50-264	R-108	07-03-67	100
Missouri	36. Rolla	U. of Missouri	Pool	50-123	R-79	11-21-61	200
	37. Columbia	U. of Missouri	Tank	50-186	R-103	10-11-66	10,000
Nebraska	38. Omaha	V.A. Hospital	TRIGA	50-131	R-57	06-26-59	18
New Mexico	39. Albuquerque	U. of New Mexico	AGN-201M No. 112	50-252	R-102	09-17-66	.005
New York	40. Buffalo	State U. of New York	PULSTAR	50-57	R-77	03-24-61	2000
	41. Sterling Forest	Union Carbide Corp.	Pool	50-54	R-81	09-07-61	5000
	42. New York City	Columbia U.	TRIGA Mark II	50-208	R-128	04-14-77	250
	43. Ithaca	Cornell U.	ZPR	50-97	R-89	12-11-62	.1
	44. Ithaca	Cornell U.	TRIGA Mk. II	50-157	R-80	01-11-62	100
	45. Riverdale	Manhattan College	Tank	50-199	R-94	03-24-64	.0001
North Carolina	46. Raleigh	North Carolina State U.	PULSTAR	50-297	R-120	08-25-72	1000
Ohio	47. Columbus	Ohio State U.	Pool	50-150	R-75	02-24-61	10
Oklahoma	48. Norman	U. of Oklahoma	AGN-211 No. 102	50-112	R-53	12-29-58	.015
Oregon	49. Corvallis	Oregon State U.	AGN-201 No. 114	50-106	R-51	11-13-58	.0001
	50. Corvallis	Oregon State U.	TRIGA Mark II	50-243	R-106	03-07-67	1000
	51. Portland	Reed Institute	TRIGA Mk. I	50-288	R-112	07-02-68	250
Pennsylvania	52. University Park	Pennsylvania State U.	TRIGA Mk. III	50-5	R-2	07-08-55	1000

NON—POWER REACTORS IN THE U.S.

RESEARCH REACTORS – CONT.

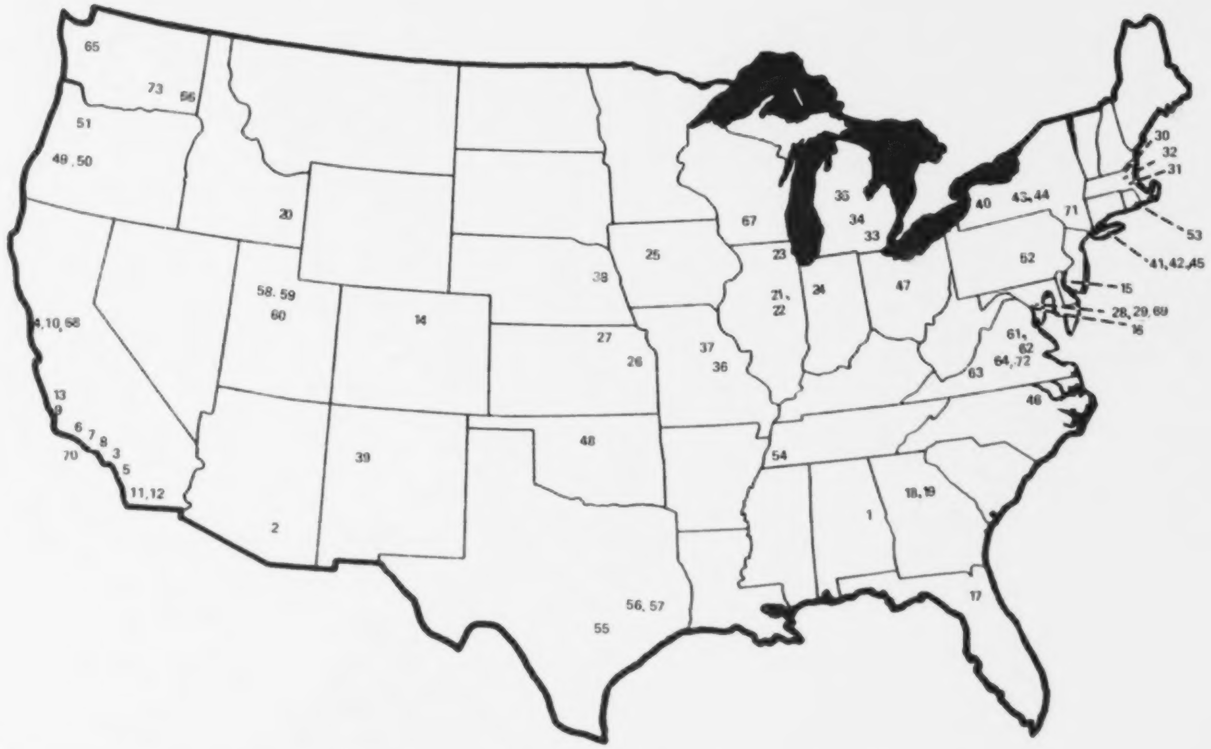
State	Location	Licensee	Reactor Type	Docket	License Number	Date Issued	Authorized Power Level (KW)
Rhode Island	53. Ft. Kearney, Narragansett	Rhode Island & Providence Plantations AEC	GE Pool	50-193	R-95	07-21-64	2000
Tennessee	54. Shelby County	Memphis State U.	AGN-201 No. 108	50-538	R-127	12-10-76	.0001
Texas	55. Austin	U. of Texas	TRIGA Mk. I	50-192	R-92	08-02-63	250
	56. College Station	Texas A&M U.	TRIGA	50-128	R-83	12-07-61	1000
	57. College Station	Texas A&M U.	AGN-201M No. 106	50-59	R-23	08-26-57	.005
Utah	58. Salt Lake City	U. of Utah	AGN-201M No. 107	50-72	R-25	09-12-57	.005
	59. Salt Lake City	U. of Utah	TRIGA Mark I	50-407	R-126	09-30-75	100
	60. Provo	Brigham Young U.	L-77	50-262	R-109	09-07-67	.01
Virgina	61. Charlottesville	U. of Virgina	Pool	50-62	R-66	06-27-60	2000
	62. Charlottesville	U. of Virginia	Cavalier	50-396	R-123	09-24-74	.1
	63. Blacksburg	Virginia Polytechnic Institute	UTR-10	50-124	R-62	12-18-59	100
	64. Lynchburg	Babcock & Wilcox Co.	LPR	50-99	R-47	09-05-58	1000
Washington	65. Seattle	U. of Washington	Argonaut	50-139	R-73	03-31-61	100
	66. Pullman	Washington State U.	TRIGA	50-27	R-76	03-06-61	1000
Wisconsin	67. Madison	U. of Wisconsin	TRIGA	50-156	R-74	11-23-60	1000

2. EXPERIMENTAL AND TEST REACTORS

State	Location	Licensee	Reactor Type	Docket	License Number	Date Issued	Authorized Power Level (MWt)
California	68. Alameda Co.	General Electric Co.	GETR	50-70	TR-1	01-07-59	50
Maryland	69. Gaithersburg	National Bureau of Standards	TEST	50-184	TR-5	11-13-67 POL 06-30-70 OL	10

3. CRITICAL EXPERIMENT FACILITIES

State	Location	Licensee	Docket	License Number	Date Issued	Experiments
California	70. Ventura Co.	Rockwell International Corp. (Possession Only)	50-147	CX-17	10-03-60	AETR Experiments (Terminal Requested).
New York	71. Schenectady	Rensselaer Polytechnic Institute	50-225	CX-22	07-03-64	
Virginia	72. Lynchburg	Babcock & Wilcox Co.	50-13	CX-10	10-22-68	NMSR Experiments
Washington	73. Richland	Battelle Memorial Institute	50-380	CX-26	11-29-71	Plutonium Recycle Critical Facility



Source: NRC Facilities License Application Record (FLAR) 06-30-77.

ABBREVIATIONS

AGN	Aerojet General Nuclear	These reactors were constructed by Aeroject at San Ramon, California, and then transferred to the current location.
ZPR	Zero Power Reactor	
LPR	Low Power Reactor	

DISTRIBUTION LIST—NRC

INTERNAL DISTRIBUTION

Office of the Commissioners	5
Atomic Safety and Licensing Appeal Panel	3
Advisory Committee on Reactor Safeguards	6
Office of Inspector and Auditor	1
Office of Policy Evaluation	1
Office of the General Counsel	1
Office of Public Affairs	21
Office of Congressional Affairs	3
Office of the Executive Director for Operations	7
Office of Administration	2
Office of the Executive Legal Director	2
Office of the Controller	2
Office for Analysis and Evaluation of Operational Data	2
Office of International Programs	3
Office of State Programs	1
Office of Management and Program Analysis	14
Office of Standards Development	1
Office of Nuclear Material Safety and Safeguards	2
– Division of Fuel Cycle and Material Safety	3
– Division of Safeguards	2
Office of Nuclear Reactor Regulation	5
– Division of Project Management	6
– Division of Operating Reactors	105
– Division of Systems Safety	4
– Division of Site Safety and Environmental Analysis	18
Office of Nuclear Regulatory Research	4
Office of Inspection and Enforcement	7
– Region I	11
– Region II	10
– Region III	10
– Region IV	10
– Region V	7
	279

EXTERNAL DISTRIBUTION

SPECIAL REQUESTS

Air Force	1
Congress	10
Department of Energy	32
Government Accounting Office	1
Environmental Protection Agency	1
Electric Power Research Institute	1
Argonne National Laboratory	3
Federal Power Commission	1
Bureau of Mines	1
Department of Agriculture	1
Department of Commerce	1
Department of the Interior	1
	54

OTHER

GPO Depository	440
GPO Stores	75
National Technical Information Service	25
Subscriptions (NTIS)	250
Colleges and Libraries (including Public Document Rooms)	128
Utilities and Other Requests	227
	1145

15/4-3

Nuclear Regulatory Commission

OPERATING UNITS STATUS REPORT

DATA AS OF 02-29-80

NUREG 0020
VOL. 4 NO. 3
MARCH 1980

NEWARK PUBLIC LIBRARY

MAY 21 1980

U. S. G[illegible]S.
DEPOSITORY 376

LICENSED OPERATING REACTORS

Y3.N88:

Available from
GPO Sales Program
Division of Technical Information and Document Control
U. S. Nuclear Regulatory Commission
Washington, D.C. 20555
and
National Technical Information Service
Springfield, Virginia 22161
703-326-8543
A Year's Subscription, 12 Issues, for this Publication is
$120.00 North American Continent ($240.00 Other).
Multiple Subscriptions Entered at the Same Time to the Same Address
Cost $90.00 North American Continent ($180.00 Other) for each Additional Subscription

Single copies are $12.50 North American Continent ($25.00 Other).

This document is provided free of charge to those who participate in the data collection program, and to the Nuclear Regulatory Commission's local public document rooms.

Issued by the U.S. Nuclear Regulatory Commission

PUBLISHED: April 21, 1980

Nuclear Regulatory Commission

OPERATING UNITS STATUS REPORT

DATA AS OF 02-29-80

NUREG 0020
VOL. 4 NO. 3
MARCH 1980

LICENSED OPERATING REACTORS

AUTHORIZATION AND CLEARANCE

THE U.S. NUCLEAR REGULATORY COMMISSION'S OFFICE OF MANAGEMENT AND PROGRAM ANALYSIS PUBLISHES THIS MONTHLY STATUS REPORT "AS PART OF THE REPORTING REQUIREMENTS IN SECTION 50.36 OF 10 CFR PART 50 UNDER GAO CLEARANCE NUMBER B-180225, WITH AN EXPIRATION DATE OF SEPTEMBER 30, 1981," AS STATED IN THE OCTOBER 3, 1978 LETTER FROM JOHN M. LOVELADY, ASSISTANT DIRECTOR, GENERAL GOVERNMENT DIVISION, U. S. GENERAL ACCOUNTING OFFICE, TO J.M. FELTON, DIRECTOR, DIVISION OF RULES AND RECORDS, U.S. NUCLEAR REGULATORY COMMISSION.

STATEMENT OF PURPOSE

The U. S. Nuclear Regulatory Commission's monthly OPERATING UNITS STATUS REPORT — LICENSED OPERATING REACTORS, provides data on the operation of nuclear units as timely and accurately as possible. This information is collected by the Office of Management and Program Analysis from the Headquarters Staff of NRC's Office of Inspection and Enforcement, from NRC's Regional Offices, and from utilities. Since all of the data concerning operation of the units is provided by the utility operators less than two weeks after the end of the month, necessary corrections to published information are shown on the ERRATA page.

This report is divided into three sections: the first contains monthly highlights and statistics for commercial operating units, and errata from previously reported data; the second is a compilation of detailed information on each unit, provided by NRC Regional Offices and IE Headquarters; and the third section is an appendix containing nuclear availability and capacity factors, forced outage rate statistics, spent fuel storage capability, reactor-years of experience and non-power reactors in the United States.

Throughout the report, statistical factors for periods greater than a report month, or for more than one unit, are computed as the arithmetic average of each unit's individual operating statistics. The statistical factors for each unit for the report month are computed from actual power production for the month. The factors for the life-span of each unit (the "Cumulative" column) are reported by the utility and are not entirely re-computed by NRC. Utility power production data is checked for consistency with previously submitted statistics.

It is hoped this status report proves informative and helpful to all agencies and individuals interested in analyzing trends in the nuclear industry which might have safety implications, or in maintaining an awareness of the U. S. energy situation as a whole.

PHILIP A. ROSS
REVIEW COORDINATOR

DEBBIE NICHOLS
ASSISTANT

TABLE OF CONTENTS

GLOSSARY ii
INDEX TO LICENSED UNITS iii

SECTION 1 – CURRENT DATA SUMMARIES

MONTHLY HIGHLIGHTS OF COMMERCIAL NUCLEAR POWER UNITS 1-2

- Licensed Power Reactors 1-2
- Power Generation 1-2
- Actual vs. Potential Energy Production 1-2
- Outage Data 1-2
- Derated Units 1-2
- Shutdowns - Number and Reasons 1-3
- Unit Availability, Capacity, and Forced Outage Rate 1-3
- Refueling Schedule 1-3

AVERAGE CAPACITY FACTORS BY VENDOR 1-4

AVERAGE DAILY POWER LEVEL FOR ALL COMMERCIAL OPERATING UNITS 1-5

ERRATA 1-7

SECTION 2 – OPERATING POWER REACTORS

ARKANSAS 1 THROUGH ZION 2 PAGES A–2 TO Z–5

- Facility Data
- Inspection and Enforcement Status
- Reports Received from Licensee
- Operating Status
- Average Daily Power Level (MWe)
- Unit Shutdowns/Reductions

SECTION 3 – APPENDIX

STATUS OF SPENT FUEL STORAGE CAPABILITY 3-2
REACTOR-YEARS OF EXPERIENCE 3-3
NON-POWER REACTORS IN THE U.S. 3-4

GLOSSARY

AVERAGE DAILY POWER LEVEL (MWe)	The net electrical energy generated during the day (measured from 0001 to 2400 hours inclusive) in megawatts hours divided by 24 hours.
LICENSED THERMAL POWER (MWt)	The maximum thermal power of the reactor authorized by the NRC, expressed in megawatts.
DATE OF COMMERCIAL OPERATION	Date unit was declared by utility owner to be available for the regular production of electricity; usually related to satisfactory completion of qualification tests as specified in the purchase contract and to accounting policies and practices of utility.
DESIGN ELECTRICAL RATING (DER) (Net MWe)	The nominal net electrical output of the unit specified by the utility and used for the purpose of plant design.
FORCED OUTAGE	An outage required to be initiated no later than the weekend following discovery of an off-normal condition.
FORCED OUTAGE HOURS	The clock hours during the report period that a unit is unavailable due to forced outages.
GROSS ELECTRICAL ENERGY GENERATED (MWH)	Electrical output of the unit during the report period as measured at the output terminals of the turbine generator, in megawatts hours.
GROSS HOURS	The clock hours from the beginning of a specified situation until its end. For outage durations, the clock hours during which the unit is not in power production.
GROSS THERMAL ENERGY GENERATED (MWH)	The thermal energy produced by the unit during the report period as measured or computed by the licensee in megawatt hours.
HOURS GENERATOR ON-LINE	Also, "Unit Service Hours." The total clock hours in the report period during which the unit operated with breakers closed to the station bus. These hours added to the total outage hours experienced by the unit during the report period, shall equal the hours in the report period.
HOURS IN REPORTING PERIOD	For units in power ascension at the end of the period, the gross hours from the beginning of the period or the first electrical production, whichever comes last, to the end of the period. For units in commercial operation at the end of the period, the gross hours from the beginning of the period or of commercial operation, whichever comes last, to the end of the period or decommissioning, whichever comes first.
HOURS REACTOR CRITICAL	The total clock hours in the report period during which the reactor sustained a controlled chain reaction.
MAXIMUM DEPENDABLE CAPACITY (GROSS) (MDC Gross) (Gross MWe)	Dependable main-unit gross capacity, winter or summer, whichever is smaller. The dependable capacity varies because the unit efficiency varies during the year due to cooling water temperature variations. It is the gross electrical output as measured at the output terminals of the turbine generator during the most restrictive seasonal conditions (usually summer).
MAXIMUM DEPENDABLE CAPACITY (NET) (MDC Net) (Net MWe)	Maximum Dependable Capacity (Gross) less the normal station service loads.
NAMEPLATE RATING (Gross MWe)	The nameplate power designation of the generator in megavolt amperes (MVA) times the nameplate power factor of the generator. NOTE: The nameplate rating of the generator may not be indicative of the maximum or dependable capacity, since some other item of equipment of a lesser rating (e.g., turbine) may limit unit output.
NET ELECTRICAL ENERGY GENERATED	Gross electrical output of the unit measured at the output terminals of the turbine generator during the reporting period, minus the normal station service electrical energy utilization. If this quantity is less than zero, a negative number should be recorded.
OUTAGE	A situation in which no electrical production takes place.
OUTAGE DATE	As reported on Appendix D of Reg. Guide 1.16, the date of the start of the outage. If continued from a previous month, report the same outage date but change "Method of Shutting Down Reactor" to "4 (Other)" and add a note: "Continued from previous month."
OUTAGE DURATION	The total clock hours of the outage measured from the beginning of the report period or the outage, whichever comes last, to the end of the report period or the outage, whichever comes first.
OUTAGE NUMBER	A number unique to the outage assigned by the licensee. The same number is reported each month in which the outage is in progress. One format is "76-05" for the fifth outage to occur in 1976.
PERIOD HOURS	See "Hours in Reporting Period."
POWER REDUCTION	A reduction in the Average Daily Power Level of more than 20% from the previous day. All power reductions are defined as outage of zero hours durations for the purpose of computing unit service and availability factors, and forced outage rate.
REACTOR AVAILABLE HOURS	The total clock hours in the report period during which the reactor was critical or was capable of being made critical. (Reactor Reserve Shutdown Hours + Hours Reactor Critical.)
REACTOR AVAILABILITY FACTOR	$\frac{\text{Reactor Available Hours} \times 100}{\text{Period Hours}}$
REACTOR RESERVE SHUTDOWN	The cessation of criticality in the reactor for administrative or other similar reasons when operation could have been continued.
REACTOR RESERVE SHUTDOWN HOURS	The total clock hours in the report period during which the reactor is in reserve shutdown mode. NOTE: No credit is given for NRC imposed shutdowns.
REACTOR SERVICE FACTOR	$\frac{\text{Hours Reactor Critical} \times 100}{\text{Period Hours}}$
REPORT PERIOD	Usually, the preceding calendar month. Can also be the preceding calendar year, (Year-to-Date), or the life-span of a unit (cumulative).
RESTRICTED POWER LEVEL	Maximum net electrical generation to which the unit is restricted during the report period due to the state of equipment, external conditions, administrative reasons, or a direction by NRC.
SCHEDULED OUTAGE	Planned removal of a unit from service for refueling, inspection, training, or maintenance. Those outages which do not fit the definition of "Forced Outage" perforce are "Scheduled Outages."
STARTUP AND POWER ASCENSION TEST PHASE	Period following initial criticality during which the unit is tested at successively higher levels, culminating with operation at full power for a sustained period and completion of warranty runs. Following this phase, the utility generally considers the unit to be available for commercial operation.
UNIT	The set of equipment uniquely associated with the reactor, including turbine generators, and ancillary equipment, considered as a single electrical energy production facility.
UNIT AVAILABLE HOURS	The total clock hours in the report period during which the unit operated on-line or was capable of such operation. (Unit Reserve Shutdown Hours + Hours Generator On-Line.)
UNIT AVAILABILITY FACTOR*	$\frac{\text{Unit Available Hours} \times 100}{\text{Period Hours}}$
UNIT CAPACITY FACTORS	
– Using Licensed Thermal Power	$\frac{\text{Gross Thermal Energy Generated} \times 100}{\text{Period Hours} \times \text{Lic. Thermal Power}}$
– Using Nameplate Rating	$\frac{\text{Gross Electrical Energy Generated} \times 100}{\text{Period Hours} \times \text{Nameplate Rating}}$
– Using DER	$\frac{\text{Net Electrical Energy Generated} \times 100}{\text{Period Hours} \times \text{DER}}$
– Using MDC Gross	$\frac{\text{Gross Electrical Energy Generated} \times 100}{\text{Period Hours} \times \text{MDC Gross}}$
– Using MDC Net	$\frac{\text{Net Electrical Energy Generated} \times 100}{\text{Period Hours} \times \text{MDC Net}}$

NOTE: If MDC Gross and/or MDC Net have not been determined, the DER is substituted for this quantity for Unit Capacity Factor calculations.

UNIT FORCED OUTAGE RATE	$\frac{\text{Forced Outage Hours}}{\text{Unit Service Hours} + \text{Forced Outage Hours}}$
UNIT RESERVE SHUTDOWN	The removal of the unit from on-line operation for economic or other similar reasons when operation could have been continued.
UNIT RESERVE SHUTDOWN HOURS	The total clock hours in the report period during which the unit was in reserve shutdown mode.
UNIT SERVICE FACTOR	$\frac{\text{Unit Service Hours} \times 100}{\text{Period Hours}}$
UNIT SERVICE HOURS	See "Hours Generator On-Line."

INDEX TO LICENSED UNITS

ARKANSAS 1 A-3
ARKANSAS 2 A-5
BEAVER VALLEY 1 B-3
BIG ROCK POINT 1 B-5
BROWNS FERRY 1 B-7
BROWNS FERRY 2 B-9
BROWNS FERRY 3 B-11
BRUNSWICK 1 B-13
BRUNSWICK 2 B-15
CALVERT CLIFFS 1 C-3
CALVERT CLIFFS 2 C-5
COOK 1 C-7
COOK 2 C-9
COOPER STATION C-11
CRYSTAL RIVER 3 C-13
DAVIS-BESSE 1 D-3
DRESDEN 1 D-5
DRESDEN 2 D-7
DRESDEN 3 D-9
DUANE ARNOLD D-11
FARLEY 1 F-3
FITZPATRICK F-5
FORT CALHOUN 1 F-7
FORT ST VRAIN F-9
GINNA G-3
HADDAM NECK H-3
HATCH 1 H-5
HATCH 2 H-7
HUMBOLDT BAY H-9
INDIAN POINT 1 I-3
INDIAN POINT 2 I-5
INDIAN POINT 3 I-7
KEWAUNEE K-3
LA CROSSE L-3
MAINE YANKEE M-3
MILLSTONE 1 M-5
MILLSTONE 2 M-7
MONTICELLO M-9
NINE MILE POINT 1 N-3
NORTH ANNA 1 N-5
OCONEE 1 O-3
OCONEE 2 O-5
OCONEE 3 O-7
OYSTER CREEK 1 O-9
PALISADES P-3
PEACH BOTTOM 2 P-5
PEACH BOTTOM 3 P-7
PILGRIM 1 P-9
POINT BEACH 1 P-11
POINT BEACH 2 P-13
PRAIRIE ISLAND 1 P-15
PRAIRIE ISLAND 2 P-17
QUAD CITIES 1 Q-3
QUAD CITIES 2 Q-5
RANCHO SECO 1 R-3
ROBINSON 2 R-5
SALEM 1 S-3
SAN ONOFRE 1 S-5
ST LUCIE 1 S-7
SURRY 1 S-9
SURRY 2 S-11
THREE MILE ISLAND 1 T-3
THREE MILE ISLAND 2 T-5
TROJAN T-7
TURKEY POINT 3 T-9
TURKEY POINT 4 T-11
VERMONT YANKEE 1 V-3
YANKEE-ROWE 1 Y-3
ZION 1 Z-3
ZION 2 Z-5

SECTION 1 – CURRENT DATA SUMMARIES

MONTHLY HIGHLIGHTS OF COMMERCIAL NUCLEAR POWER UNITS

FOR MONTH OF

FEBRUARY 1980

LICENSED POWER REACTORS:

NUMBER		CAPACITY [1] MWe (Net)
66	IN COMMERCIAL OPERATION	48,135
1	NOT IN COMMERCIAL OPERATION	912
67 [2]	LICENSED TO OPERATE	49,047
1 [3]	LICENSED FOR FUEL LOADING AND LOW POWER TESTING	

POWER GENERATION	REPORT MONTH	PREVIOUS MONTH	YEAR-TO-DATE
1. GROSS ELECTRICAL (MWHE)	20,287,108	20,039,154	40,326,262
2. NET ELECTRICAL (MWHE)	19,331,056	19,087,131	38,418,187
3. AVG. UNIT SERVICE FACTOR (%)	63.1	61.6	62.4
4. AVERAGE UNIT AVAILABILITY FACTOR (%)	63.1	62.0	62.6
5. AVERAGE UNIT CAPACITY FACTOR (MDC) (%)	56.4	54.6	55.5
6. AVG. UNIT CAPACITY FACTOR (DER) (%)	54.9	53.3	54.1
7. FORCED OUTAGE RATE (%)	10.4	8.5	9.6

NOTES

[1] BASED UPON MAXIMUM DEPENDABLE CAPACITY; DESIGN ELECTRIC RATING USED IF MDC NOT DETERMINED.

[2] EXCLUDES INDIAN POINT 1 (DER = 265), HUMBOLDT BAY (DER = 65), AND THREE MILE ISLAND 2 (DER = 906) WHICH HAVE OPERATING LICENSES BUT ARE SHUTDOWN INDEFINITELY.

[3] SEQUOYAH 1 (DER = 1129) WAS LICENSED ON FEBRUARY 29, 1980 FOR FUEL LOADING AND LOW POWER TESTING ONLY.

ACTUAL VS. POTENTIAL ENERGY PRODUCTION

			% OF POTENTIAL PRODUCTION
POTENTIAL ENERGY PRODUCTION IN THIS PERIOD BY UNITS IN COMMERCIAL OPERATION (Using Maximum Dependable Capacity Net)		33,468,552 MWHe	
ENERGY ACTUALLY PRODUCED DURING THIS REPORT PERIOD	=	19,331,056 MWHe NET	57.7
ENERGY NOT PRODUCED DURING THIS REPORT PERIOD DUE TO SCHEDULED OUTAGES (NET)	=	8,723,005 MWHe	26.1
ENERGY NOT PRODUCED DURING THIS REPORT PERIOD DUE TO FORCED OUTAGES (NET)	=	3,076,751 MWHe	9.2
ENERGY NOT PRODUCED DURING THIS REPORT PERIOD FOR OTHER REASONS (NET)	=	2,337,739 MWHe	7.0
TOTAL		33,468,551 MWHe =	100%
ENERGY NOT PRODUCED THIS REPORT PERIOD DUE TO NRC IMPOSED SHUTDOWNS AND RESTRICTIONS	=	2,276,614 MWHe	6.8
NUMBER OF UNITS WITH POWER RESTRICTIONS IMPOSED BY NRC	=	1	

OUTAGE DATA

	NUMBER	HOURS	PERCENT OF CLOCK TIME	MWHE LOST PRODUCTION
1. FORCED OUTAGES DURING MONTH	58	4,140.8	9.0	3,076,751
2. SCHEDULED OUTAGES DURING THE MONTH	39	12,826.7	27.9	8,723,005
3. TOTAL	97	16,967.5	36.9	11,799,757

MWHE LOST PRODUCTION = Down time x maximum dependable capacity net

DERATED UNITS

UNIT NAME	MAXIMUM DEPENDABLE CAPACITY (MWe - NET)	POWER LIMIT (MWe - NET)
BIG ROCK POINT 1	63	63
CALVERT CLIFFS 1	810	790
†FORT ST. VRAIN	330	231
INDIAN POINT 2	864	834
INDIAN POINT 3	965	917
POINT BEACH 1	495	390
ROBINSON 2	2300 MWT	2200 MWT

†Restricted by NRC

MONTHLY HIGHLIGHTS OF COMMERCIAL NUCLEAR POWER UNITS

FOR MONTH OF

FEBRUARY 1980

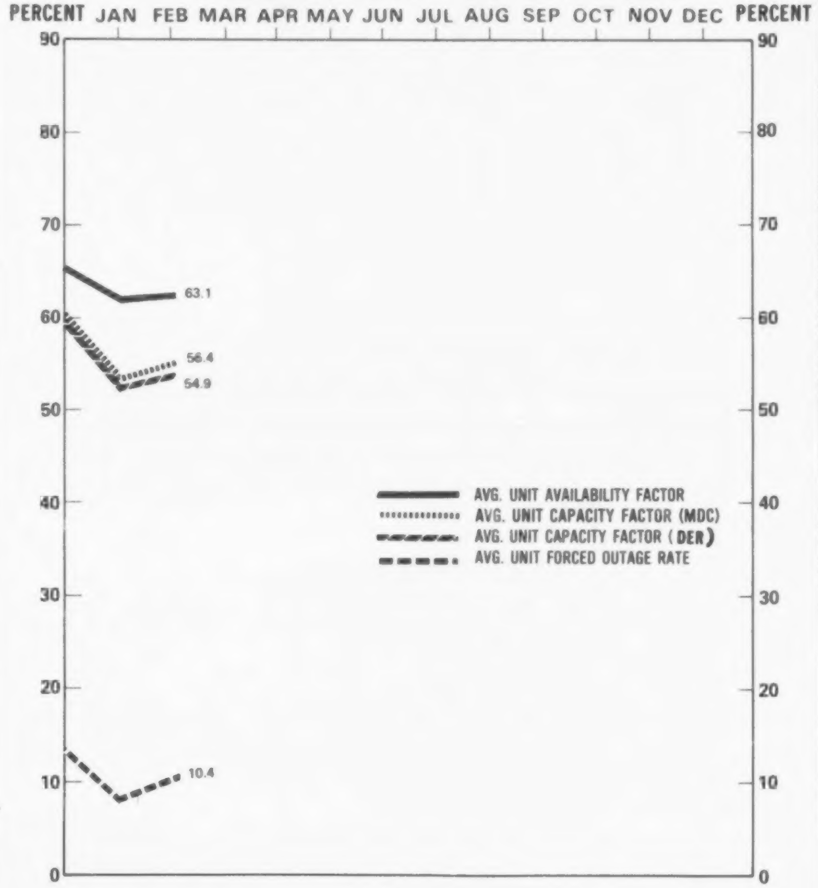
REASONS FOR SHUTDOWN

REASON	NUMBER	TOTAL HOURS LOST
A - Equipment Failure	33	1,803.1
B - Maintenance or Test	18	1,542.5
C - Refueling	16	9,115.8
D - NRC Restriction	11	3,576.0
E - Operator Training and License Examination	1	18.2
F - Administrative	0	0
G - Operational Error	8	92.7
H - Other	10	819.2
TOTAL	97	16,967.5

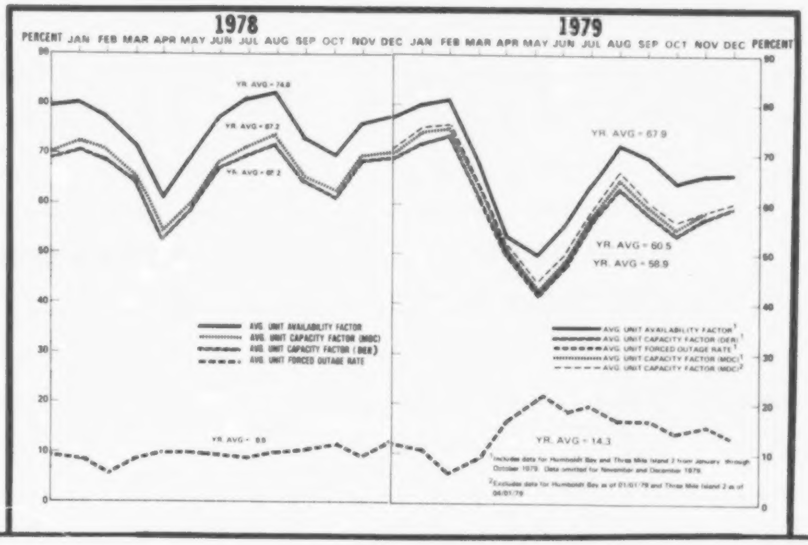
SHUTDOWNS GREATER THAN 72 HOURS EACH

UNIT NAME	REASON	UNIT NAME	REASON
ARKANSAS 1	D	NORTH ANNA 1	H
BEAVER VALLEY 1	C	OCONEE 1	A,2B
BROWNS FERRY 1	C	OYSTER CREEK 1	C
BROWNS FERRY 2	H	PALISADES	D
CALVERT CLIFFS 1	D	PEACH BOTTOM 3	B
CRYSTAL RIVER 3	A	PILGRIM 1	C
DAVIS-BESSE 1	A	PRAIRIE ISLAND 1	B
DRESDEN 1	C	PRAIRIE ISLAND 2	C
DRESDEN 3	C	QUAD CITIES 2	C
DUANE ARNOLD	C	RANCHO SECO 1	C
FARLEY 1	D	SAN ONOFRE 1	H
FITZPATRICK	A	SURRY 1	D
FORT CALHOUN 1	C	SURRY 2	C
FORT ST VRAIN	B	THREE MILE ISLAND 1	D
INDIAN POINT 2	A,D	TURKEY POINT 3	C
INDIAN POINT 3	C	TURKEY POINT 4	A
LA CROSSE	A	VERMONT YANKEE 1	D
MAINE YANKEE	C	YANKEE-ROWE 1	D
MONTICELLO	C	ZION 1	2A

UNIT AVAILABILITY, CAPACITY AND FORCED OUTAGE



UNIT AVAILABILITY, CAPACITY AND FORCED OUTAGE

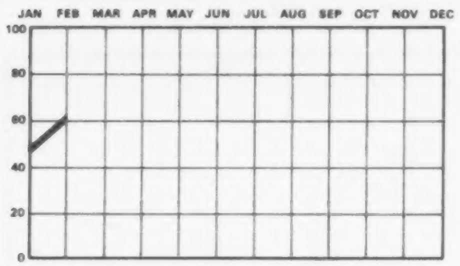


REFUELING SCHEDULE

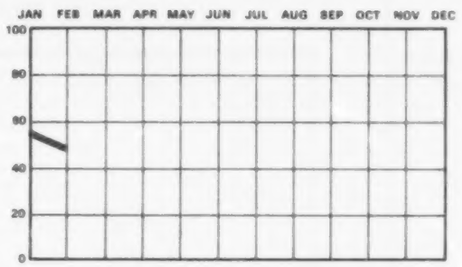
Please refer to the table on page 3–6 for the scheduled refueling data for each nuclear powerplant in this publication.

VENDOR CAPACITY FACTORS

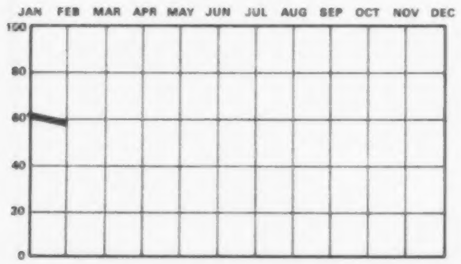
WESTINGHOUSE



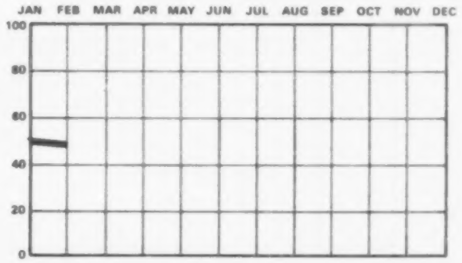
COMBUSTION ENGINEERING



GENERAL ELECTRIC



BABCOCK & WILCOX



FEBRUARY 1980

OPERATIONS

1. COMPUTED BY ARITHMETIC AVERAGE OF UNIT CAPACITY FACTORS

GE BWR's	CFMDC
BROWNS FERRY 1	.0
BROWNS FERRY 2	38.3
BROWNS FERRY 3	97.9
BRUNSWICK 1	95.7
BRUNSWICK 2	80.8
COOPER STATION	91.7
DRESDEN 1	.0
DRESDEN 2	93.2
DRESDEN 3	2.2
DUANE ARNOLD	22.1
FITZPATRICK	59.9
HATCH 1	75.5
HATCH 2	94.8
MILLSTONE 1	76.1
MONTICELLO	44.7
NINE MILE POINT 1	84.6
OYSTER CREEK 1	.0
PEACH BOTTOM 2	84.9
PEACH BOTTOM 3	81.4
PILGRIM 1	.0
QUAD CITIES 1	89.1
QUAD CITIES 2	.0
VERMONT YANKEE 1	87.8

23 UNITS

W PWR's	CFMDC
BEAVER VALLEY 1	.0
COOK 1	97.5
COOK 2	91.5
FARLEY 1	33.5
GINNA	101.0
HADDAM NECK	98.9
INDIAN POINT 2	31.7
INDIAN POINT 3	8.5
KEWAUNEE	98.1
NORTH ANNA 1	70.8
POINT BEACH 1	72.3
POINT BEACH 2	89.4
PRAIRIE ISLAND 1	76.1
PRAIRIE ISLAND 2	22.7
ROBINSON 2	99.5
SALEM 1	76.5
SAN ONOFRE 1	59.3
SURRY 1	62.6
SURRY 2	.0
TROJAN	89.8
TURKEY POINT 3	67.8
TURKEY POINT 4	78.3
YANKEE-ROWE 1	.0
ZION 1	14.0
ZION 2	94.4

25 UNITS

CE PWR's	CFMDC
CALVERT CLIFFS 1	40.4
CALVERT CLIFFS 2	99.8
FORT CALHOUN 1	0.0
MAINE YANKEE	0.0
MILLSTONE 2	88.0
PALISADES	0.0
ST LUCIE 1	100.4

7 UNITS

B&W PWR's	CFMDC
ARKANSAS 1	62.3
CRYSTAL RIVER 3	80.5
DAVIS-BESSE 1	76.9
OCONEE 1	1.5
OCONEE 2	72.0
OCONEE 3	97.0
RANCHO SECO 1	.0
THREE MILE ISLAND 1	.0

8 UNITS

2. COMPUTED BY RATIO OF VENDOR TOTALS

	GE BWR's	W PWR's	CE PWR's	B&W PWR's	All PWR's
NET ELECTRICAL PRODUCTION	7,156,005	7,942,536	1,868,473	2,297,430	12,108,439
MDC NET	17,351	18,428	5,178	6,737	30,343
CFMDC	59.3	61.9	51.8	49.0	57.3

Capacity factor in this page, denoted as CFMDC, is a function of the net maximum dependable capacity. See the corresponding definition in the Glossary. The vendor averages are computed by the formula :

$$\frac{\text{Net Electrical Energy Produced by Vendor}}{\text{Potential Electrical Production By Vendor In This Month}} \times 100\%$$

Units Excluded	Reason for Exclusion
Indian Point 1 Humboldt Bay Three Mile Island 2	Unit Shutdown Indefinitely
Arkansas 2	Not in Commercial Operation
Big Rock Point	Not Representative of Typical BWRs
LaCrosse	Not Representative of Typical BWRs and Atypical Vendor
Fort St. Vrain	Atypical Vendor and Reactor Design

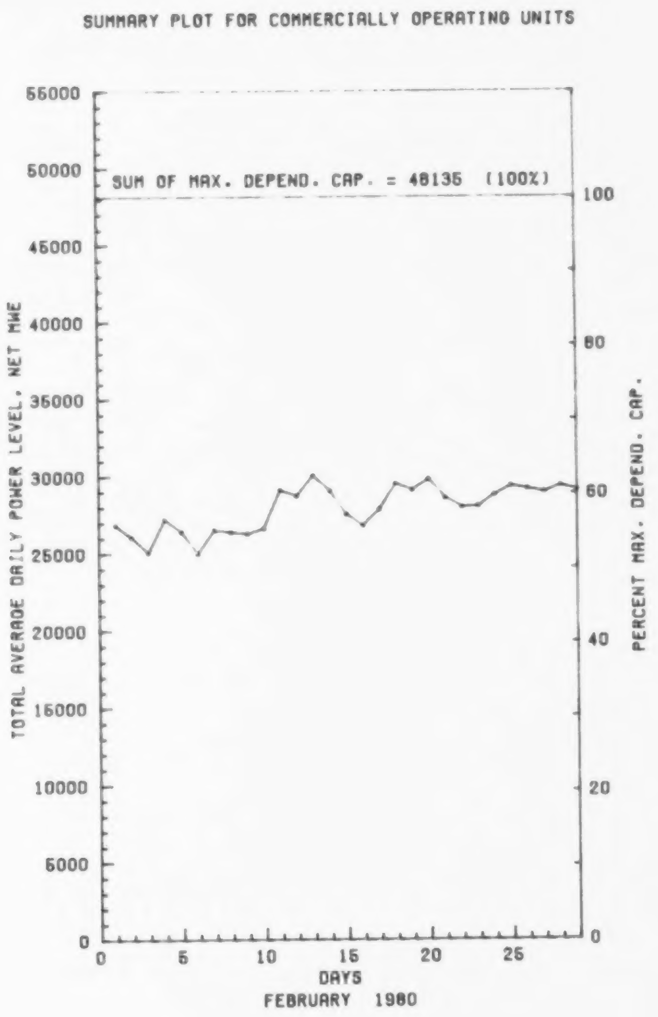
AVERAGE DAILY POWER LEVEL FOR ALL COMMERCIALLY OPERATING UNITS

This chart depicts the average daily power level for the units in commercial operation during the month.

The straight line on the graph labelled "SUM OF MAX DEP CAP'S" is plotted at the value shown by summing the separate maximum dependable capacities of the commercially operating units (in Net MWe). The plot shown below the line is calculated by summing the separate average daily power levels of the commercially operating units for each day of the month.

The scale on the left vertical axis runs in 1,000 MWe increments from 0 to 55,000 MWe (Net). The right vertical axis shows the percentage in 10% increments, up to 100% of the "SUM OF MAX DEP CAP'S."

It should be recognized that the 100% line would be obtainable only if all of the commercially operating units operated at 100% capacity, 24 hours per day, for the entire month. In other words, since any power generator must occasionally shut down to refuel and/or perform needed maintenance, and also since 100% capacity production is not always required by power demands, the 100% line is a theoretical goal and not a practical one.



ERRATA

Corrections and Amendments to Previously-Reported Data

(NOTE: These changes are reflected in the data contained in the current report)

CORRECTIONS TO NUREG 0020

Page	Unit Name	Item	Correction
	NONE		

MULTIPLE - ISSUE CORRECTIONS

NONE

SECTION 2 – OPERATING POWER REACTORS

ARKANSAS 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Arkansas/Pope 6 miles NW of Russelville
2.	Electric Reliability Council	Member of SW Power Pool
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2568
5.	Design Electrical Rating (MWE Net)	850
6.	Date of Initial Criticality	August 6, 1974
7.	Date of Commercial Operation	December 19, 1974
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Dardenelle Reservoir

Utility & Contractor Information

10.	Licensee	Arkansas Power and Light Company
11.	Corporate Address	P.O. Box 551 Little Rock, Arkansas 72203
12.	Corporate Contact	W. Cavanaugh III Vice Pres., Generation & Construction
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	B&W
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	IV
17.	IE Branch Chief	G. L. Madsen
18.	IE Senior Inspector	T. F. Westerman
19.	IE Resident Inspector	W. D. Johnson (501) 968-3290
20.	Licensing Project Manager	G. S. Vissing
21.	Docket Number	50-313
22.	License Number/Date of Issue	DPR-51, 5/21/74
23.	Public Document Room	Arkansas Polytechnic College Russelville, Arkansas 72801

INSPECTION STATUS

ENFORCEMENT STATUS

FAILURE TO FOLLOW APPENDIX B TO 10 CFR 50 AND PROCEDURE 1004.10, TEST CONTROL IN THAT NO PROCEDURES EXIST FOR TESTING OF PROTECTIVE RELAYS ON ESF 4.16KV SWITCHGIAR AND NO SCHEDULE EXISTS FOR TESTING OF THESE RELAY DEVICES.
(8001 INFRACTION)

CONTRARY TO CRITERION XVII AND APL-TOP-1A PARAGRAPH 11.4.2 THE TEST RECORD SHEETS FOR PROTECTIVE RELAYING ON THE ESF 4.16KV SWITCHGEAR DO NOT HAVE ACCEPTANCE CRITERION AND ARE NOT SIGNED BY THE COGNIZANT SUPERVISOR AS ASSIGNED BY EXHIBIT I TO 1004.10.
(8001 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

+ NUMBER 2 LOW PRESSURE TURBINE HAS DISKS REMOVED DUE TO STRESS CRACKING DISCOVERED DURING THE LAST OUTAGE. GENERATOR OUTPUT IS REDUCED BY ABOUT 10%.

FACILITY ITEMS (PLANS AND PROCEDURES):

MANAGERIAL ITEMS

PLANT STATUS:

+ THE UNIT IS OPERATING AT 90% POWER.

LAST IE SITE INSPECTION DATE: JANUARY 22 - FEBRUARY 21, 1980 +

INSPECTION REPORT NO: 50-313/80-03 +

SUMMARY

INSPECTION SUMMARY

+ INSPECTION CONDUCTED DURING THE PERIOD OF JANUARY 7-18, 1980 (80-01): ROUTINE, UNANNOUNCED INSPECTION OF PREVIOUSLY IDENTIFIED ITEMS, AND CONTROL OF DESIGN AND DESIGN CHANGES AND MODIFICATIONS. WITHIN THE TWO AREAS INSPECTED, TWO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED (INFRACTION & DEFICIENCY - FAILURE TO COMPLY WITH APPENDIX B TO 10 CFR 50.)

INSPECTION CONDUCTED DURING PERIOD OF DECEMBER 22, 1979 - JANUARY 21 1980 (80-02): ROUTINE, ANNOUNCED INSPECTION OF PLANT PROCEDURES AND ROUTINE PLANT OPERATIONS. WITHIN THE TWO AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED.

INSPECTION CONDUCTED DURING THE PERIOD OF JANUARY 22 - FEBRUARY 21, 1930 (80-03): ROUTINE, ANNOUNCED INSPECTION OF PLANT OPERATIONS, TEST AND MEASURING EQUIPMENT PROGRAM, FOLLOW-UP ON PREVIOUSLY IDENTIFIED ITEMS, IMPLEMENTATION OF TMI LESSONS LEARNED REQUIREMENTS, FOLLOW-UP ON 10 CFR PART 21 REPORTS, AND MAINTENANCE. THE INSPECTION INVOLVED 149 INSPECTOR-HOURS ON-SITE BY FOUR NRC INSPECTORS. WITHIN THE SIX AREAS INSPECTED, ONE ITEM OF NONCOMPLIANCE WAS IDENTIFIED (INFRACTION - FAILURE TO CONTROL ACCESS TO A HIGH RADIATION AREA.)

REPORTS RECEIVED FROM LICENSEE

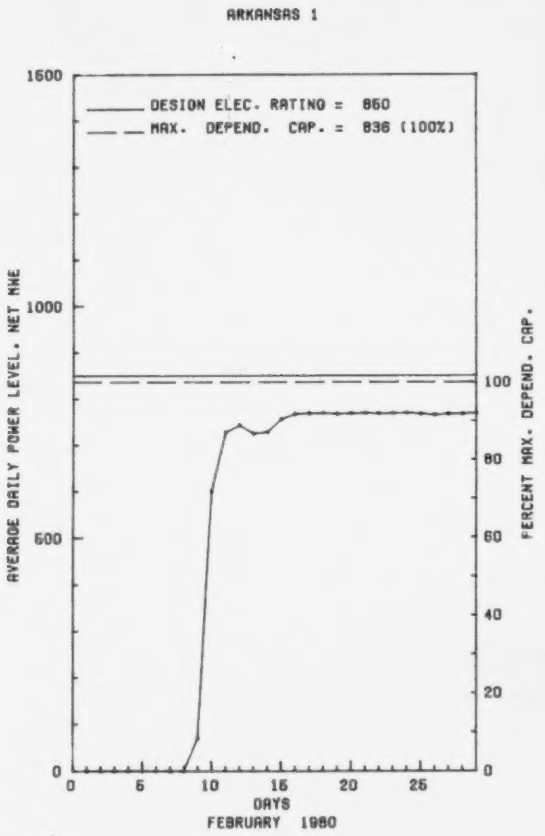
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-01 03-L0	1/14/80	2/13/80	HYDROGEN PURGE SUPPLY ISOLATION VALVE FAILURE
80-02 03-L0	1/14/80	2/13/80	HYDROGEN PURGE EXHAUST ISOLATION VALVE FAILURE
80-03 03-L0	1/21/80	2/4/80	CONTAINMENT SPRAY PIPE CRACKS

REVIEWED BY OMPA LOEB: PE Bobe DATE: 4/10/80

REVIEWED BY NRR DOR: Greg S Vissing DATE: 4/11/80

ARKANSAS 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-313
2. Utility Contact: L. S. Bramlett (501) 968-2519
3. Licensed Thermal Power (MWt): 2568
4. Nameplate Rating (Gross MWe): 1003 x 0.9 = 903
5. Design Electrical Rating (Net MWe): 850
6. Maximum Dependable Capacity (Gross MWe): 883
7. Maximum Dependable Capacity (Net MWe): 836
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	696.0	1,440.0	45,571.0
12. Hours Reactor Was Critical	493.3	493.3	30,631.5
13. Reactor Reserve Shutdown Hours	.0	.0	4,247.4
14. Hours Generator On-Line	488.8	488.8	29,959.3
15. Unit Reserve Shutdown Hours	.0	.0	756.7
16. Gross Thermal Energy (MWH)	1,295,311	1,295,311	72,467,255
17. Gross Electrical Energy (MWH)	379,570	379,570	24,101,726
18. Net Electrical Energy (MWH)	362,342	362,342	22,996,378
19. Unit Service Factor	70.2	33.9	65.7
20. Unit Availability Factor	70.2	33.9	67.5
21. Unit Capacity Factor (MDC Net)	62.3	30.1	56.4
22. Unit Capacity Factor (DER Net)	61.2	29.6	59.6
23. Unit Forced Outage Rate	.0	.0	15.4

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
79-06	12/31/79	S	207.2	D	4		CH	INSTRU	COMMITTMENT TO NRC TO PROVIDE TMI II RELATED MODIFICATIONS.

SUMMARY

THE UNIT REMAINED SHUTDOWN UNTIL 02/09/80 FOR TMI-RELATED MODIFICATIONS. THE LOW PRESSURE TURBINE "B" WAS MODIFIED DUE TO CRACKS IN THE TURBINE ROTOR DURING THE OUTAGE. BECAUSE OF THIS, THE UNIT OPERATED UNDER A WESTINGHOUSE-IMPOSED RESTRICTION OF 780 MWE (GROSS) UNTIL 02/15/80. A BRIEF LOAD REDUCTION OCCURRED ON 02/14/80 WHEN THE NO. 1 GOVERNOR VALVE STEM ON THE HIGH PRESSURE TURBINE FAILED. OPERATION WAS ROUTINE THE REMAINDER OF THE REPORT, WITH POWER LIMITED TO 805 MWE BECAUSE OF THE MODIFICATION TO THE LOW PRESSURE TURBINE "B" BLOCKING.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

ARKANSAS 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Arkansas/Pope 6 miles NW of Russelville
2.	Electric Reliability Council	Member of SW Power Pool
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2815
5.	Design Electrical Rating (MWE Net)	912
6.	Date of Initial Criticality	12-5-78
7.	Date of Commercial Operation	N/A
8.	Condenser Cooling Method	Recirculating
9.	Condenser Cooling Water Source	Cooling Tower

Utility & Contractor Information

10.	Licensee	Arkansas Power & Light Company
11.	Corporate Address	P.O. Box 551 Little Rock, Arkansas 72203
12.	Corporate Contact	W. Cavanaugh III Vice Pres., Generation & Construction
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Combustion Engineering, Inc.
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	IV
17.	IE Branch Chief	G. L. Madsen
18.	IE Senior Inspector	T. F. Westerman
19.	IE Resident Inspector	W. D. Johnson (501) 968-3290
20.	Licensing Project Manager	G. S. Vissing
21.	Docket Number	50-368
22.	License Number/Date of Issue	NFP-6, 9/1/78
23.	Public Document Room	Arkansas Polytechnic College Russelville, Arkansas 72801

INSPECTION STATUS

ENFORCEMENT STATUS

FAILURE TO FOLLOW APPENDIX B TO 10 CFR 50 AND PROCEDURE 1004.10, TEST CONTROL, IN THAT NO PROCEDURES EXIST FOR TESTING OF PROTECTIVE RELAYS ON ESF 4.16KV SWITCHGEAR AND NO SCHEDULE EXISTS FOR TESTING OF THESE RELAY DEVICES.
(8001 INFRACTION)

CONTRARY TO CRITERION XVII AND APL-TOP-1A PARAGRAPH 11.4.2 THE TEST RECORD SHEETS FOR PROTECTIVE RELAYING ON THE ESF 4.16KV SWITCHGEAR DO NOT MEET ACCEPTANCE CRITERION AND ARE NOT SIGNED BY THE COGNIZANT SUPERVISOR ASSIGNED BY EXHIBIT I TO 1004.10.
(8001 DEFICIENCY)

CONTRARY TO TECHNICAL SPECIFICATION 6.8.1 THE LICENSEE FAILED TO FOLLOW APPENDIX DD, STEP 2.3, OF PROCEDURE 2.800.01, "POWER ASCENSION TEST," IN THAT THE CEAC'S WERE NOT RETURNED TO SERVICE AS REQUIRED AFTER THE DROPPED CEA WAS RETURNED TO THE FULLY WITHDRAWN POSITION.
(8002 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NO. 1 DIESEL GENERATOR OUT OF COMMISSION DUE TO FAILED BEARING BETWEEN DIESEL AND GENERATOR THAT OVERHEATED SHAFT CAUSING SHAFT TO SHEAR.

FACILITY ITEMS (PLANS AND PROCEDURES):

MANAGERIAL ITEMS:

PLANT STATUS:

PLANT IS IN COLD SHUTDOWN. RECOVERY FROM SHUTDOWN FOR IMPLEMENTING NUREG 0578 REQUIREMENTS DELAYED DUE TO BEARING AND SHAFT FAILURE ON NO. 1 DIESEL GENERATOR.

LAST IE INSPECTION DATE: JANUARY 22 - FEBRUARY 21, 1980

INSPECTION REPORT NO: 50-368/80-03

SUMMARY

INSPECTION SUMMARY

INSPECTION CONDUCTED DURING THE PERIOD OF JANUARY 7-18, 1980 (80-01): ROUTINE, UNANNOUNCED INSPECTION OF PREVIOUSLY IDENTIFIED ITEMS, AND CONTROL OF DESIGN AND DESIGN CHANGES AND MODIFICATIONS. WITHIN THE TWO AREAS INSPECTED, TWO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED (INFRACTION & DEFICIENCY - FAILURE TO COMPLY WITH APPENDIX B TO 10 CFR 50.)

INSPECTION CONDUCTED DURING PERIOD OF DECEMBER 22, 1979 - JANUARY 21, 1980 (80-02): ROUTINE, ANNOUNCED INSPECTION OF PLANT PROCEDURES AND POWER ASCENSION TESTING. WITHIN THE TWO AREAS INSPECTED ONE ITEM OF NONCOMPLIANCE WAS IDENTIFIED (INFRACTION - FAILURE TO ADHERE TO PROCEDURE.)

INSPECTION CONDUCTED DURING PERIOD OF JANUARY 22, 1980 - FEBRUARY 21, 1980 (80-03): ROUTINE, ANNOUNCED INSPECTION OF PLANT OPERATIONS REVIEW, TEST AND MEASURING EQUIPMENT PROGRAM, FOLLOW-UP ON PREVIOUSLY IDENTIFIED ITEMS, IMPLEMENTATION OF TMI LESSONS LEARNED REQUIREMENTS, FOLLOW-UP ON 10 CFR PART 21 REPORTS, MAINTENANCE, POWER ASCENSION TEST WITNESSING, REVIEW OF POWER ASCENSION TEST DATA, AND REVIEW OF LICENSEE'S AUTHORIZATION TO RAISE POWER. THE INSPECTION INVOLVED 206 INSPECTOR-HOURS ON-SITE BY FOUR NRC INSPECTORS. WITHIN THE NINE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

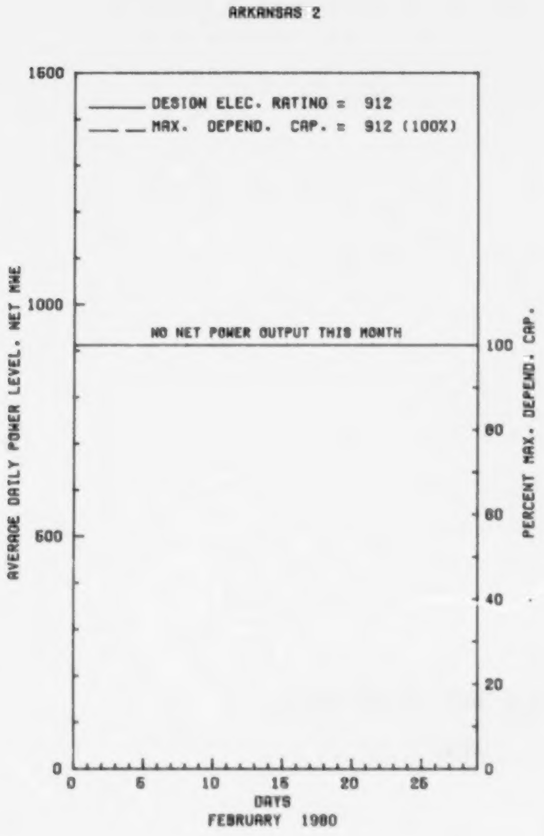
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-01 03-L0	1/7/80	2/5/80	CONTAINMENT ISOLATION VALVE FAILURE
80-02 03-L0	1/10/80	2/5/80	CPC CHANNEL FAILURE
80-03 03-L0	1/20/80	2/15/80	EFW FLOW CONTROL VALVE FAILURE
80-04 03-L0	1/7/80	2/15/80	CPC CHANNEL FAILURE
80-05 03-L0	1/23/80	2/15/80	REACTOR COOLANT SYSTEM LEAKAGE
80-06 03-L0	1/23/80	2/15/80	CPC CHANNEL FAILURE

REVIEWED BY OMPA LOEB: **DATE:** 04/10/80

REVIEWED BY NRR DOR: **DATE:** 4/11/80

ARKANSAS 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-368
2. Utility Contact: Rex Pendergraft (501) 968-2519
3. Licensed Thermal Power (MWt): 2815
4. Nameplate Rating (Gross MWe): 959
5. Design Electrical Rating (Net MWe): 912
6. Maximum Dependable Capacity (Gross MWe): 912
7. Maximum Dependable Capacity (Net MWe): 912
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	696.0	1,440.0	10,344.6
12. Hours Reactor Was Critical	.0	691.0	4,519.4
13. Reactor Reserve Shutdown Hours	.0	.0	2,293.1
14. Hours Generator On-Line	.0	628.1	3,665.0
15. Unit Reserve Shutdown Hours	.0	.0	21.7
16. Gross Thermal Energy (MWH)	0	1,322,956	4,819,777
17. Gross Electrical Energy (MWH)	0	431,827	1,418,112
18. Net Electrical Energy (MWH)	0	410,234	1,296,063
19. Unit Service Factor	.0	.0	.0
20. Unit Availability Factor	.0	.0	
21. Unit Capacity Factor (MDC Net)	.0		.0
22. Unit Capacity Factor (DER Net)		.0	.0
23. Unit Forced Outage Rate	.0	.0	.0

UNIT NOT IN COMMERCIAL OPERATION

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 03/10/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-04	01/29/80	S	696.0	B	4				SCHEDULED REACTOR TRIP PER S/U TESTING SEQUENCE. DURING OUTAGE, MODIFICATIONS WERE MADE RELATED TO TMI LESSONS LEARNED. DURING A SURVEILLANCE TEST ON #2 DIESEL GENERATOR, ROTOR SHAFT FAILED EXTENDING OUTAGE.

SUMMARY

THE UNIT REMAINED SHUTDOWN THROUGHOUT THE REPORT PERIOD FOR TMI-RELATED MODIFICATIONS. WHILE TESTING THE EMERGENCY DIESEL GENERATOR, A SHEARED GENERATOR ROTOR SHAFT WAS DISCOVERED AT THE INBOARD BEARING COUPLING. THIS WAS SHIPPED TO FAIRBANKS-MORSE FOR REPAIRS.

LEGEND

[1] Type:
- F–Forced
- S–Scheduled

[2] Reason:
- A–Equipment Failure
- B–Maintenance or Test
- C–Refueling
- D–Regulatory Restriction
- E–Operator Training & License Examination
- F–Administrative
- G–Operational Error
- H–Other

[3] Method:
- 1–Manual
- 2–Manual Scram
- 3–Automatic Scram
- 4–Continuations
- 5–Load Reduction
- 9–Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

NRC POWER RESTRICTIONS: NONE

BEAVER VALLEY 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Pennsylvania/Beaver 5 miles E of E. Liverpool, Ohio
2.	Electric Reliability Council	ECAR
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2652
5.	Design Electrical Rating (MWE Net)	852
6.	Date of Initial Criticality	May 10, 1976
7.	Date of Commercial Operation	10/01/76 (50%) 04/30/77 (100%)
8.	Condenser Cooling Method	Cooling tower/Closed loop
9.	Condenser Cooling Water Source	Ohio River

Utility & Contractor Information

10.	Licensee	Duquesne Light Company
11.	Corporate Address	435 Sixth Avenue Pittsburgh, Pennsylvania 15219
12.	Corporate Contact	C. N. Dunn, Vice President
13.	Architect/Engineer	Stone & Webster
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Senior Inspector	R. R. Keimig
19.	IE Resident Inspector	D. Beckman (412) 643-9200
20.	Licensing Project Manager	D. L. Wigginton
21.	Docket Number	50-334
22.	License Number/Date of Issue	DPR-66, 1/30/76
23.	Public Document Room	B. F. Jones Memorial Library 633 Franklin Avenue Aliquippa, PA 15001

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO TECHNICAL SPECIFICATION 6.8.1.A, REGULATORY GUIDE 1.33, NOVEMBER 1972, THE BVPS OPERATING MANUAL SECTION 1.43.4, REVISION 4, PROCEDURES FOR THE OPERATION AND CALIBRATION OF PROCESS RADIATION MONITORING EQUIPMENT WERE NOT PROPERLY ESTABLISHED AND IMPLEMENTED, IN THAT: 1) RADCON MANUAL FORM NO. RCM-F229 INCLUDED INCORRECT ACCEPTANCE CRITERIA FOR THE SAMPLE FLOW RATES REQUIRED TO BE MAINTAINED ON SEVEN GASEOUS AND/OR PARTICULATE RADIATION MONITORS AND WAS THE DOCUMENT IN USE FOR PERIODIC CHECKS AND ADJUSTMENT OF THESE FLOWRATES; 2) DATA COLLECTED ON FORM RCM-F229 DURING THE PERIOD OF OCTOBER 3 - NOVEMBER 28, 1979 DOCUMENTS OPERATION OF EIGHT RADIATION MONITORS AT SAMPLE FLOW RATES WHICH DID NOT MEET THEIR RESPECTIVE ACCEPTANCE CRITERIA WITH NO CORRECTIVE ACTION CONSISTENTLY IMPLEMENTED; AND, 3) NEITHER PROCEDURES NOR APPROPRIATE SETPOINTS HAVE BEEN ESTABLISHED FOR THE CALIBRATION AND MAINTENANCE OF RADIATION MONITOR SAMPLE FLOW ALARM EQUIPMENT.
(7924 INFRACTION)

CONTRARY TO 10 CFR 50, APPENDIX B, CRITERION VI, BVPS FSAR, APPENDIX A, SECTION A.2.2.6; QA PROCEDURE OP-8, REVISION 4, SECTION 3.3.4; AND THE BVPS MAINTENANCE MANUAL, CHAPTER 1, SECTION 7, REVISION 6. ON AUGUST 28, 1979 CALIBRATION OF RCS HOT LEG RTD AMPLIFIER TM-RC-432H WAS PERFORMED UTILIZING DATA FROM AN INSTRUMENT CALIBRATION DATA SHEET, FORM 147, WHICH HAD NOT BEEN REVIEWED, APPROVED, OR DISTRIBUTED AS A REVISION OR ON-THE-SPOT CHANGE TO MAINTENANCE SURVEILLANCE PROCEDURE NO. 6.40 IN ACCORDANCE WITH THE ABOVE REQUIREMENTS.
(7924 INFRACTION)

ON NOVEMBER 27, 1979, **APPROVED A SWITCHING ORDER** FOR A MAINTENANCE WORK REQUEST WITHOUT VERIFYING THAT ALL REDUNDANT ENGINEERED SAFETY FEATURE SYSTEMS WERE AVAILABLE IN THAT THE APPROVED MAINTENANCE WORK RENDERED BOTH ECCS SUBSYSTEMS INOPERABLE FROM APPROXIMATELY 8:30 A.M. TO 10:30 A.M.
(7930 VIOLATION)

ON NOVEMBER 27, 1979, **APPROVED AN EQUIPMENT** CLEARANCE PERMIT ASSOCIATED WITH A MAINTENANCE WORK REQUEST WITHOUT VERIFYING THAT ALL REDUNDANT ENGINEERED SAFETY FEATURE SYSTEMS WERE AVAILABLE IN THAT THE APPROVED CLEARANCE PERMIT RENDERED BOTH ECCS SUBSYSTEMS INOPERABLE FROM APPROXIMATELY 8:30 A.M. TO 10:30 A.M.
(7930 VIOLATION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

FACILITY ITEMS (PLANS AND PROCEDURES):

- Refueling/modification outage commenced on 11/30/79. The outage is expected to last until 7/80.

MANAGERIAL ITEMS:

A partial reorganization of station and corporate departments began on 12/16/79, and is substantially complete, pending further personnel assignments and approval of revised Tech. Specs.

PLANT STATUS:

- Refueling completed on 3/1/80. In-service inspection, modifications and maintenance will continue through current outage.

- LAST IE SITE INSPECTION DATE: 3/7/80

- INSPECTION REPORT NO: 50-334/80-06

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-148/ 03L	12/10/79	02/07/80	30 Day LER	Dessicant bag in engine mounted fuel tank blocked suction line to diesel electric fuel pump
80-002/ 01T	01/17/80	01/30/80	14 Day LER	While in mode 5 with RCS level at mid-span in the loops suction was lost to all RHR pumps due to air binding
80-004/ 03L	01/23/80	02/07/80	30 Day LER	Radiation monitor setpoints less conservative than required for refueling but corrected prior to removing head from vessel
80-005/ 03L	01/12/80	02/07/80	30 Day LER	Operator entered restricted area without having rad tech or ratemeter with him
80-006/ 04T	01/30/80	02/07/80	14 Day LER	Tritium grab sample in Ohio River near plant discharge of 4240 picocuries/liter (1390 is reporting level) with no tritium releases above 10 CFR 201 limits

SUMMARY

INSPECTION SUMMARY

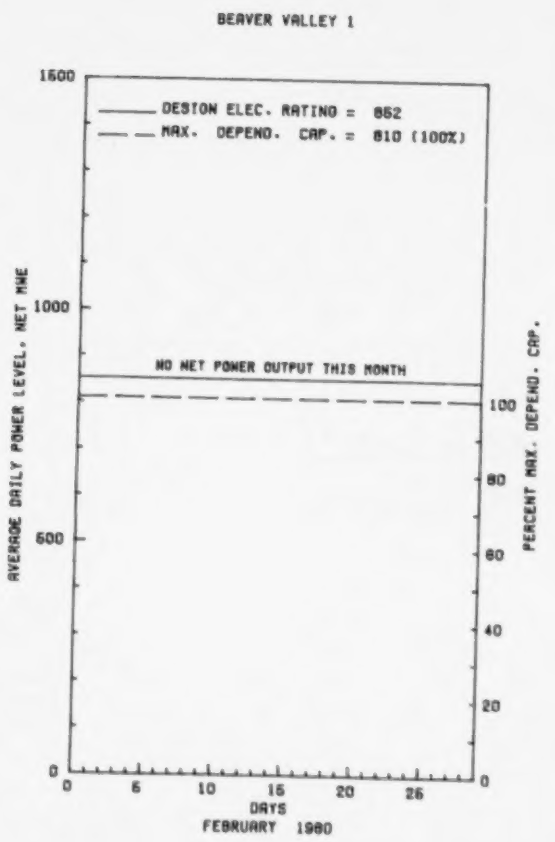
- 50-334/80-02 - Jan 8-11: Routine, unannounced inspection by one region-based inspector (29.5 hours). Inspection areas (3): inservice inspection (ISI) activities including ISI program review, review of implementing NDE procedures and ISI data review; activities associated with design change modifications; and licensee action on previous inspection findings. Noncompliances (1) (failure to review and approve NDE procedure prior to its implementation).
- 50-334/80-04 - Jan 22: Special public enforcement meeting convened by the Director, Office of Inspection and Enforcement, to discuss the licensee's actions pursuant to an Order Modifying License issued on December 5, 1979. The meeting was attended by senior licensee and NRC:OIE management and the public. A subsequent question and answer session was held to provide NRC responses to public concerns.

REVIEWED BY OMPA LOEB: T. C. Cintula **DATE:** 4·10·80

REVIEWED BY NRR DOR: D L Wigginton **DATE:** 4. 11 80

BEAVER VALLEY 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-334
2. Utility Contact: DAVID R. TIMKO (412) 643-5308
3. Licensed Thermal Power (MWt): 2652
4. Nameplate Rating (Gross MWe): 1026 x 0.9 = 923
5. Design Electrical Rating (Net MWe): 852
6. Maximum Dependable Capacity (Gross MWe): 845
7. Maximum Dependable Capacity (Net MWe): 810
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: Item 7 decreased; was not based on most restrictive season.
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	696.0	1,440.0	33,600.0
12. Hours Reactor Was Critical	0	0	13,774.7
13. Reactor Reserve Shutdown Hours	0	0	3,482.7
14. Hours Generator On-Line	0	0	13,105.0
15. Unit Reserve Shutdown Hours	0	0	0
16. Gross Thermal Energy (MWH)	0	0	26,9[illegible],253
17. Gross Electrical Energy (MWH)	0	0	8,277,940
18. Net Electrical Energy (MWH)	0	0	7,570,743
19. Unit Service Factor	0	0	41.9
20. Unit Availability Factor	0	0	41.9
21. Unit Capacity Factor (MDC Net)	0	0	32.0
22. Unit Capacity Factor (DER Net)	0	0	31.2
23. Unit Forced Outage Rate	0	0	46.4

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 07/22/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	License Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
1	12/01/79	S	696.0	C	4		ZZ	ZZZZZZ	UNIT SHUTDOWN FOR MAJOR MODIFICATIONS AND REFUELING OUTAGE. STARTUP SCHEDULED FOR JULY, 1980.

SUMMARY

THE UNIT REMAINED SHUTDOWN THROUGHOUT THE REPORT PERIOD FOR A MAJOR MODIFICATION AND REFUELING OUTAGE.

LEGEND

[1] Type:
F-Forced
S-Scheduled

[2] Reason:
A-Equipment Failure
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error
H-Other

[3] Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Continuations
5-Load Reduction
9-Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

BIG ROCK POINT 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Michigan/Charlevoix 4 miles NE of Charlevoix
2.	Electric Reliability Council	ECAR
3.	Type of Reactor	BWR
4.	Power Level (MWT)	240
5.	Design Electrical Rating (MWE Net)	72
6.	Date of Initial Criticality	September 27, 1962
7.	Date of Commercial Operation	March 29, 1963
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10.	Licensee	Consumers Power Company
11.	Corporate Address	212 West Michigan Avenue Jackson, Michigan 49201
12.	Corporate Contact	R. B. DeWitt
13.	Architect/Engineer	Bechtel Corporation
14.	Nuclear Steam Supply System	GE
15.	Constructor	Bechtel Corporation

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Senior Inspector	D.C. Boyd
19.	IE Principal Inspector	G. Wright
20.	Licensing Project Manager	R. D. Silver
21.	Docket Number	50-155
22.	License Number/Date of Issue	DPR-6 issued 8/30/62
23.	Public Document Room	Charlevoix Public Library 107 Clinton Street Charlevoix, Michigan

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

FACILITY ITEMS

+ The Plant was shutdown the first two weeks of January 1980 to complete short term lessons learned commitments, resuming operations on January 14. (80-01)

MANAGERIAL ITEMS

+ The shift technical advisor system is in place. Operations crews are now on a five shift schedule which allows one extra week on day-shift for training. (80-01)

PLANT STATUS

Currently operating at 208 Mwt. The plant was shutdown during the first two weeks of January to make required short term lessons learned changes of NUREG 0578.

+LAST IE SITE INSPECTION DATE: January 8-11, 1980

+ INSPECTION REPORT NO: 80-01

SUMMARY

INSPECTION SUMMARY

+ Inspection on January 8-11, (80-01): Routine, unannounced inspection of the licensee's onsite Quality Assurance Program changes; Procedure System; and followup actions relative to Licensee Event Reports, IE Bulletins, IE Circulars, and open inspection items. The inspection involved 50 inspector hours onsite by two NRC inspectors. No items of noncompliance were identified in the six areas inspected.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

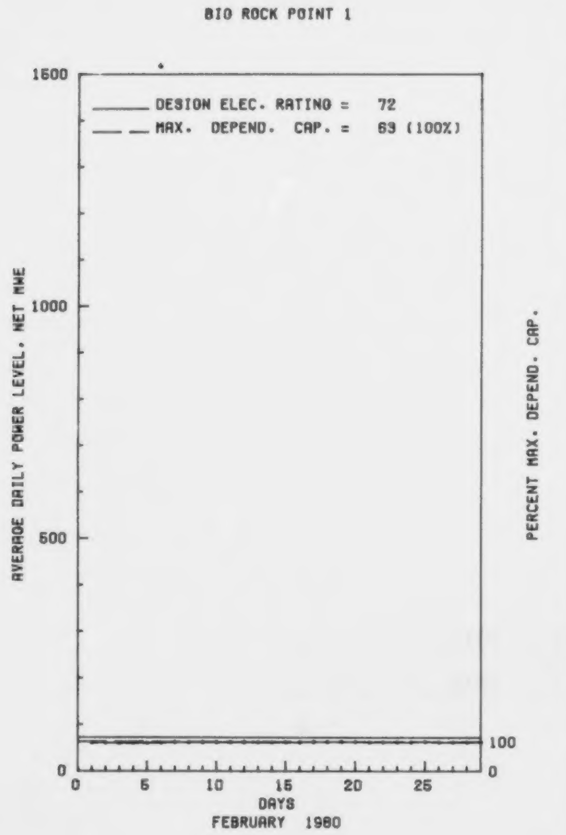
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
80-01/03L-0	01/14/80	02/12/80	30 Day LER	Observation of temperature data for loop 2 of the Emergency Condenser Sys. reveals that seat leakage on normally closed outlet valve MO7053 is occurring.
80-02/03L-0	01/24/80	02/21/80	30 Day LER	Cell 14 of the battery for channel C of the Reactor Depressurizing System read 1.191.
80-04/01T-0	02/01/80	02/12/80	14 Day LER	Following observation of minor temp. and pressure changes in reactor cooling water auxiliary systems.

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 4-10-80

REVIEWED BY NRR DOR: **DATE:**

BIG ROCK POINT 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-155
2. Utility Contact: G. Himebauch (616) 547-6537 x180
3. Licensed Thermal Power (MWt): 240
4. Nameplate Rating (Gross MWe): 70.6 x 0.85 = 60
5. Design Electrical Rating (Net MWe): 72
6. Maximum Dependable Capacity (Gross MWe): 67
7. Maximum Dependable Capacity (Net MWe): 63
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: MDC NET DECREASED TO 63-FUEL THERMAL HYDRAU LIMIT MARGINS
9. Power Level To Which Restricted, If Any (Net MWe): 63
10. Reasons for Restrictions, If Any: FUEL THERMAL HYDRAULIC LIMITS

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	696.0	1,440.0	148,363.0
12. Hours Reactor Was Critical	696.0	1,118.2	101,251.2
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	696.0	1,105.5	99,104.4
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	149,313	230,348	18,346,279
17. Gross Electrical Energy (MWH)	45,601	70,315	5,804,233
18. Net Electrical Energy (MWH)	43,107	66,375	5,454,712
19. Unit Service Factor	100.0	76.8	66.8
20. Unit Availability Factor	100.0	76.8	66.8
21. Unit Capacity Factor (MDC Net)	98.3	73.2	54.6
22. Unit Capacity Factor (DER Net)	86.0	64.0	51.1
23. Unit Forced Outage Rate	.0	3.4	25.3

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	NONE								

SUMMARY

OPERATION WAS ROUTINE THROUGHOUT THE REPORT PERIOD AT 63 MWE (NET) DUE TO FUEL THERMAL HYDRAULIC LIMITS, WITH NO SHUTDOWNS OR REDUCTIONS REPORTED.

LEGEND

[1] Type:
F-Forced
S-Scheduled

[2] Reason:
A-Equipment Failure
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error
H-Other

[3] Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Continuations
5-Load Reduction
9-Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

BROWNS FERRY 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Alabama/Limestone 10 miles NW of Decatur
2.	Electric Reliability Council	SERC
3.	Type of Reactor	BWR
4.	Power Level (MWT)	3293
5.	Design Electrical Rating (MWE Net)	1065
6.	Date of Initial Criticality	August 17, 1973
7.	Date of Commercial Operation	August 1, 1974
8.	Condenser Cooling Method	Once Through With Forced Draft Cooling Towers
9.	Condenser Cooling Water Source	Tennessee River

Utility & Contractor Information

10.	Licensee	Tennessee Valley Authority
11.	Corporate Address	500A Chestnut Street Tower II Chattanooga, Tennessee 37401
12.	Corporate Contact	H. G. Parris, Manager of Power.
13.	Architect/Engineer	TVA
14.	Nuclear Steam Supply System	GE
15.	Constructor	TVA

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. Lewis
18.	IE Senior Inspector	H. C. Dance
19.	IE Resident Inspector	R. Sullivan (205) 729-6196
20.	Licensing Project Manager	R. Clark
21.	Docket Number	50-259
22.	License Number/Date of Issue	DPR-33, 6/26/73
23.	Public Document Room	Athens Public Library South and Forrest Athens, Alabama 35611

INSPECTION STATUS

ENFORCEMENT STATUS

TECHNICAL SPECIFICATION 6.3.A.6 REQUIRES THAT DETAILED WRITTEN PROCEDURES FOR SURVEILLANCE AND TESTING BE PREPARED, APPROVED, AND ADHERED TO. SECTION 500 OF THE BFNP RADIOCHEMICAL LABORATORY MANUAL REQUIRES AN ANNUAL CHECK OF ACTIVITY AND EFFICIENCY FACTORS FOR THE GE(LI) GAMMA SPECTROSCOPY SYSTEMS. CONTRARY TO THE ABOVE, NO EFFICIENCY AND ACTIVITY CHECKS WERE MADE DURING THE PERIOD FROM FEBRUARY 1977 TO DECEMBER 1979.
(7944 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

+ NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

+ REFUELING UNTIL MARCH 15, 1980.

+ LAST IE SITE INSPECTION DATE: FEBRUARY 26-29, 1980

+ INSPECTION REPORT NO: 50-259/80-10

SUMMARY

INSPECTION SUMMARY

+ INSPECTION DECEMBER 17-20 (79-44): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 10 INSPECTOR-HOURS ON SITE IN THE AREAS OF CONFIRMATORY MEASUREMENTS INCLUDING: REVIEW OF RADIOCHEMICAL INSTRUMENT CALIBRATION AND QUALITY CONTROL; REVIEW OF RADIOCHEMICAL PROCEDURES; AND COMPARISON OF RESULTS OF THE ANALYSES OF SPLIT SAMPLES ANALYZED BY THE LICENSEE AND THE NRC RII MOBILE LABORATORY. OF THE THREE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN TWO AREAS; ONE ITEM OF NONCOMPLIANCE WAS FOUND IN ONE AREA (DEFICIENCY - FAILURE TO PERFORM ANNUAL EFFICIENCY AND ACTIVITY CHECKS FOR GE(LI) COUNTING SYSTEMS).

INSPECTION JANUARY 16-18 (80-01): THIS UNANNOUNCED INSPECTION INVOLVED 16 INSPECTOR-HOURS ON SITE IN THE AREAS OF ADMINISTRATIVE CONTROLS, THERMAL AND CHEMICAL DISCHARGES, CHEMICAL USAGE, LICENSEE EVENT REPORTS AND ENVIRONMENTAL SURVEILLANCE. OF THE FIVE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION JANUARY 21-24 (80-02): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 10 INSPECTOR-HOURS ON SITE IN THE AREAS OF OBSERVATION OF THE REFUELING ACTIVITIES OF UNIT 1 IN PREPARATION FOR CYCLE 4 AND REVIEW OF THE STARTUP TESTING ACTIVITIES CONDUCTED ON UNIT 3 AT THE START OF CYCLE 3. OF THE TWO AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION JANUARY 29-31 (80-03): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 4 INSPECTOR-HOURS ON SITE IN THE AREA OF REFUELING ACTIVITIES. OF THE ONE AREA INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION JANUARY 21 (80-04): MANAGEMENT MEETING TO DISCUSS THE IMPLEMENTATION OF REQUIREMENTS DIRECTED IN THE ORDER MODIFYING THE LICENSES OF THE BROWNS FERRY NUCLEAR PLANT, UNIT NOS. 1, 2 AND 3. THE MEETING INVOLVED 4 INSPECTOR-HOURS BY 6 NRC INSPECTORS. WITHIN THE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION FEBRUARY 5-7 (80-07): THIS SPECIAL, ANNOUNCED INSPECTION INVOLVED 4 INSPECTOR-HOURS ON SITE IN THE AREAS OF VIDEO (TV) OBSERVATION OF CORE SPRAY SYSTEM SPARGER (UNIT 1); REVIEW OF METHODS OF WELD FITUP INSPECTION (UNITS 1, 2 AND 3); AND REVIEW OF PROPOSED ACTIONS CONCERNING A LICENSEE EVENT REPORT (UNIT 1). OF THE THREE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

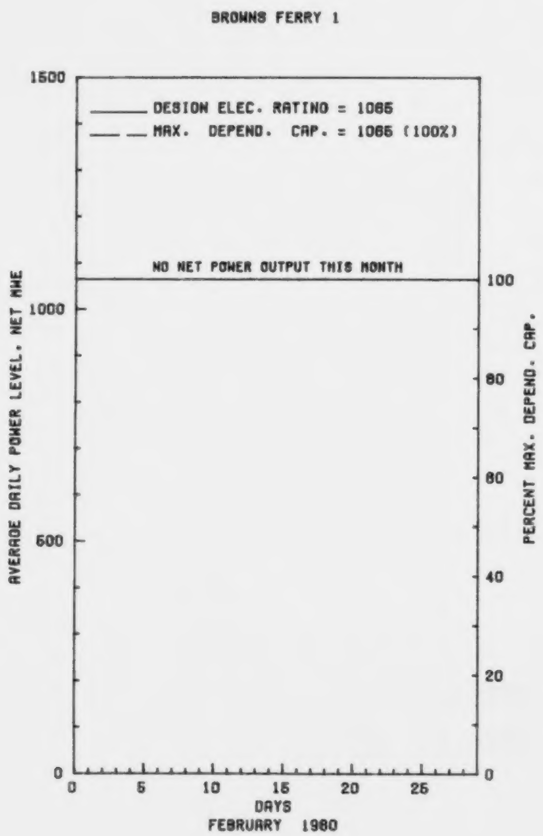
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-036/ 03L-0	12/26/79	01/24/80	30 DAY LER	DRYWELL AIR MONITOR INOPERABLE.
80-002/ 03L-0	01/12/80	02/08/80	30 DAY LER	WIND INDICATOR LIGHNING DAMAGE.
80-003/ 03L-0	01/12/80	02/04/80	30 DAY LER	MAIN STEAM ISOLATION VALVES EXCESSIVE LEAKAGE ON TEST.
80-004/ 01T-0	01/18/80	01/31/80	14 DAY LER	STAINLESS ELBOW CRACK INDICATION.

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 4-10-80

REVIEWED BY NRR DOR: Dick Clark **DATE:** 4-11-80

BROWNS FERRY 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-259
2. Utility Contact: Don Green (205) 729-6846
3. Licensed Thermal Power (MWt): 3293
4. Nameplate Rating (Gross MWe): 1280 x 0.9 = 1152
5. Design Electrical Rating (Net MWe): 1065
6. Maximum Dependable Capacity (Gross MWe): 1098
7. Maximum Dependable Capacity (Net MWe): 1065
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	696.0	1,440.0	48,938.0
12. Hours Reactor Was Critical	.0	71.7	28,349.0
13. Reactor Reserve Shutdown Hours	.0	.0	4,629.5
14. Hours Generator On-Line	.0	71.7	27,684.6
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	184,152	74,320,645
17. Gross Electrical Energy (MWH)	0	61,950	24,580,420
18. Net Electrical Energy (MWH)	0	57,164	23,859,132
19. Unit Service Factor	.0	5.0	56.6
20. Unit Availability Factor	.0	5.0	56.6
21. Unit Capacity Factor (MDC Net)	.0	3.7	45.8
22. Unit Capacity Factor (DER Net)	.0	3.7	45.8
23. Unit Forced Outage Rate	.0	.0	32.9

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 03/07/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
105	01/03/80	S	695.0	C	4				REFUELING CONTINUES.

SUMMARY

THE UNIT REMAINED SHUTDOWN THROUGHOUT THE REPORT PERIOD FOR REFUELING.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

BROWNS FERRY 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1. Location (state/county) (nearest population center)	Alabama/Limestone 10 miles NW of Decatur
2. Electric Reliability Council	SERC
3. Type of Reactor	BWR
4. Power Level (MWT)	3293
5. Design Electrical Rating (MWE Net)	1065
6. Date of Initial Criticality	July 20, 1974
7. Date of Commercial Operation	March 1, 1975
8. Condenser Cooling Method	Once Through With Forced Draft Cooling Towers
9. Condenser Cooling Water Source	Tennessee River

Utility & Contractor Information

10. Licensee	Tennessee Valley Authority
11. Corporate Address	500A Chestnut Street Tower II Chattanooga, Tennessee 37401
12. Corporate Contact	H. G. Parris, Manager of Power
13. Architect/Engineer	TVA
14. Nuclear Steam Supply System	GE
15. Constructor	TVA

Regulatory Information

16. IE Region responsible	II
17. IE Branch Chief	R. C. Lewis
18. IE Senior Inspector	H. C. Dance
19. IE Resident Inspector	R. Sullivan (205) 729-6196
20. Licensing Project Manager	R. Clark
21. Docket Number	50-260
22. License Number/Date of Issue	DPR-52, 6/28/74
23. Public Document Room	Athens Public Library South and Forrest Athens, Alabama 35611

INSPECTION STATUS

ENFORCEMENT STATUS

TECHNICAL SPECIFICATION 6.3.A.6 REQUIRES THAT DETAILED WRITTEN PROCEDURES FOR SURVEILLANCE AND TESTING BE PREPARED, APPROVED, AND ADHERED TO. SECTION 500 OF THE BFNP RADIOCHEMICAL LABORATORY MANUAL REQUIRES AN ANNUAL CHECK OF ACTIVITY AND EFFICIENCY FACTORS FOR THE GE(LI) GAMMA SPECTROSCOPY SYSTEMS. CONTRARY TO THE ABOVE, NO EFFICIENCY AND ACTIVITY CHECKS WERE MADE DURING THE PERIOD FROM FEBRUARY 1977 TO DECEMBER 1979.
(7944 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

+ NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

PLANT SHUTDOWN FOR MAINTENANCE

+ LAST IE SITE INSPECTION DATE: FEBRUARY 26-29, 1980

+ INSPECTION REPORT NO: 50-260/80-09

SUMMARY

INSPECTION SUMMARY

+ INSPECTION DECEMBER 17-20 (79-44): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 10 INSPECTOR-HOURS ON SITE IN THE AREAS OF CONFIRMATORY MEASUREMENTS INCLUDING: REVIEW OF RADIOCHEMICAL INSTRUMENT CALIBRATION AND QUALITY CONTROL; REVIEW OF RADIOCHEMICAL PROCEDURES; AND COMPARISON OF RESULTS OF THE ANALYSES OF SPLIT SAMPLES ANALYZED BY THE LICENSEE AND THE NRC RII MOBILE LABORATORY. OF THE THREE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN TWO AREAS; ONE ITEM OF NONCOMPLIANCE WAS FOUND IN ONE AREA (DEFICIENCY - FAILURE TO PERFORM ANNUAL EFFICIENCY AND ACTIVITY CHECKS FOR GE(LI) COUNTING SYSTEMS).

INSPECTION JANUARY 16-18 (80-01): THIS UNANNOUNCED INSPECTION INVOLVED 16 INSPECTOR-HOURS ON SITE IN THE AREAS OF ADMINISTRATIVE CONTROLS, THERMAL AND CHEMICAL DISCHARGES, CHEMICAL USAGE, LICENSEE EVENT REPORTS AND ENVIRONMENTAL SURVEILLANCE. OF THE FIVE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION JANUARY 21-24 (80-02): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 11 INSPECTOR-HOURS ON SITE IN THE AREAS OF OBSERVATION OF THE REFUELING ACTIVITIES OF UNIT 1 IN PREPARATION FOR CYCLE 4 AND REVIEW OF THE STARTUP TESTING ACTIVITIES CONDUCTED ON UNIT 3 AT THE START OF CYCLE 3. OF THE TWO AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION JANUARY 29-31 (80-03): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 4 INSPECTOR-HOURS ON SITE IN THE AREA OF REFUELING ACTIVITIES. OF THE ONE AREA INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION JANUARY 21 (80-04): MANAGEMENT MEETING TO DISCUSS THE IMPLEMENTATION OF REQUIREMENTS DIRECTED IN THE ORDER MODIFYING THE LICENSES OF THE BROWNS FERRY NUCLEAR PLANT, UNIT NOS. 1, 2 AND 3. THE MEETING INVOLVED 4 INSPECTOR-HOURS BY 6 NRC INSPECTORS. WITHIN THE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION FEBRUARY 5-7 (80-07): THIS SPECIAL, ANNOUNCED INSPECTION INVOLVED 5 INSPECTOR-HOURS ON SITE IN THE AREAS OF VIDEO (TV) OBSERVATION OF CORE SPRAY SYSTEM SPARGER (UNIT 1); REVIEW OF METHODS OF WELD FITUP INSPECTION (UNITS 1, 2 AND 3); AND REVIEW OF PROPOSED ACTIONS CONCERNING A LICENSEE EVENT REPORT (UNIT 1). OF THE THREE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

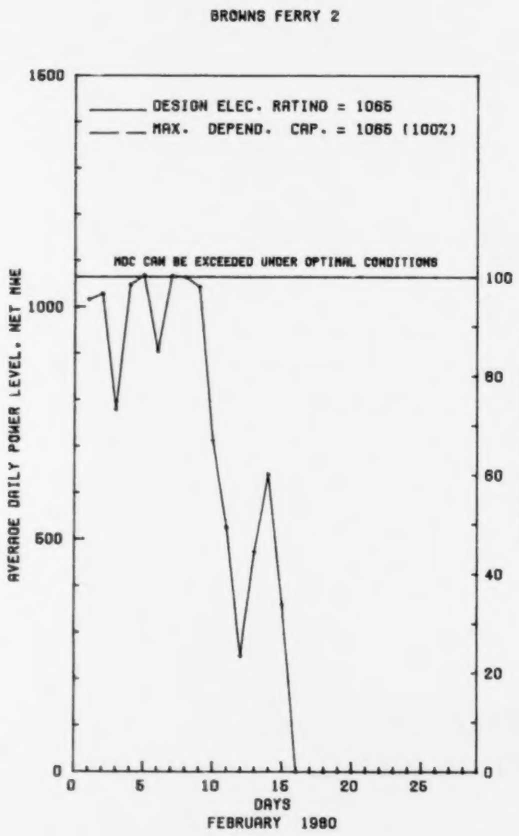
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-026/ 03L-0	12/23/79	01/18/80	30 DAY LER	DRYWELL/TORUS DIFFERENTIAL PRESSURE INDICATOR INOPERABLE.
80-001/ 03L-0	12/29/79	01/25/80	30 DAY LER	REACTOR WATER LEVEL SWITCH INOPERABLE.
80-002/ 03L-0	12/22/79	01/18/80	30 DAY LER	RECIRCULATION FLOW CONTROL VALVE INOPERABLE.

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 4-10-80

REVIEWED BY NRR DOR: Dick Clark **DATE:** 4-11-80

BROWNS FERRY 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-260
2. Utility Contact: Don Green (205) 729-6846
3. Licensed Thermal Power (MWt): 3293
4. Nameplate Rating (Gross MWe): 1280 x 0.9 = 1152
5. Design Electrical Rating (Net MWe): 1065
6. Maximum Dependable Capacity (Gross MWe): 1098
7. Maximum Dependable Capacity (Net MWe): 1065
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	696.0	1,440.0	43,949.0
12. Hours Reactor Was Critical	358.0	1,102.0	25,586.4
13. Reactor Reserve Shutdown Hours	337.9	337.9	12,112.6
14. Hours Generator On-Line	321.5	1,065.5	24,730.0
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	901,327	2,999,438	69,154,438
17. Gross Electrical Energy (MWH)	295,280	1,001,450	22,951,512
18. Net Electrical Energy (MWH)	283,856	972,837	22,394,306
19. Unit Service Factor	46.2	74.0	56.4
20. Unit Availability Factor	46.2	74.0	56.4
21. Unit Capacity Factor (MDC Net)	38.3	63.4	47.8
22. Unit Capacity Factor (DER Net)	38.3	63.4	47.8
23. Unit Forced Outage Rate	53.8	26.0	36.4

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 03/03/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
117	02/01/80	F	.0	A	5				DERATED FOR TESTS ON "C" RX FEEDPUMP TURBINE.
118	02/03/80	F	.0	A	5				DERATED - "B" RECIRC PUMP TRIPPED.
119	02/06/80	F	.0	A	5				DERATED - "B" RECIRC PUMP TRIPPED AND "A" RECIRC PUMP DISCHARGE VALVE CLOSED.
120	02/06/80	F	.0	A	5				DERATED WHEN "A" RECIRC PUMP TRIPPED.
121	02/09/80	S	.0	B	5				DERATED FOR TURBINE CV TESTS AND SI'S.
122	02/10/80	F	10.1	H	3				RPS CHANNEL TRIP.
123	02/12/80	F	17.9	H	3				RPS CHANNEL TRIP.
124	02/15/80	F	346.5	H	3				UNIT REMAINED DOWN FOR MAINTENANCE TO HPCI.

SUMMARY

THE UNIT EXPERIENCED SEVERAL RECIRCULATION PUMP TRIPS AND REACTOR PROTECTION SYSTEM CHANNEL TRIPS BEFORE BEING SHUTDOWN ON 02/15/80 FOR MAINTENANCE TO THE HIGH PRESSURE COOLANT SYSTEM.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

BROWNS FERRY 3

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Alabama/Limestone 10 miles NW of Decatur
2.	Electric Reliability Council	SERC
3.	Type of Reactor	BWR
4.	Power Level (MWT)	3293
5.	Design Electrical Rating (MWE Net)	1065
6.	Date of Initial Criticality	August 8, 1976
7.	Date of Commercial Operation	March 1, 1977
8.	Condenser Cooling Method	Once Through With Forced Draft Cooling Tower
9.	Condenser Cooling Water Source	Tennessee River

Utility & Contractor Information

10.	Licensee	Tennessee Valley Authority
11.	Corporate Address	500A Chestnut Street Tower II Chattanooga, Tennessee 37401
12.	Corporate Contact	H. G. Parris, Manager of Power
13.	Architect/Engineer	TVA
14.	Nuclear Steam Supply System	GE
15.	Constructor	TVA

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R.C. Lewis
18.	IE Senior Inspector	H. C. Dance
19.	IE Resident Inspector	R. Sullivan (205) 729-6196
20.	Licensing Project Manager	R. Clark
21.	Docket Number	50-296
22.	License Number/Date of Issue	DPR-68, 7/2/76
23.	Public Document Room	Athens Public Library South and Forrest Athens, Alabama 35611

INSPECTION STATUS

ENFORCEMENT STATUS

TECHNICAL SPECIFICATION 6.3.A.6 REQUIRES THAT DETAILED WRITTEN PROCEDURES FOR SURVEILLANCE AND TESTING BE PREPARED, APPROVED, AND ADHERED TO. SECTION 500 OF THE BFNP RADIOCHEMICAL LABORATORY MANUAL REQUIRES AN ANNUAL CHECK OF ACTIVITY AND EFFICIENCY FACTORS FOR THE GE(LI) GAMMA SPECTROSCOPY SYSTEMS. CONTRARY TO THE ABOVE, NO EFFICIENCY AND ACTIVITY CHECKS WERE MADE DURING THE PERIOD FROM FEBRUARY 1977 TO DECEMBER 1979.
(7944 DEFICIENCY)

CONTRARY TO TS 6.3.A.1 AND CRITERION V OF APPENDIX B TO 10 CFR 50, AS IMPLEMENTED BY THE ACCEPTED QUALITY ASSURANCE PROGRAM, WRITTEN PROCEDURES WERE NOT PREPARED COVERING REMOVAL AND INSTALLATION OF PRIMARY CONTAINMENT HATCHES. THIS RESULTED IN AN IMPROPERLY SECURED EQUIPMENT HATCH WHICH PREVENTED MAINTAINING PRIMARY CONTAINMENT INTEGRITY ON DECEMBER 6-9, 1979.
(7945 VIOLATION)

CONTRARY TO 10 CFR 50.36(C)(2) THE REACTOR WAS NOT SHUTDOWN WHEN LOSS OF NITROGEN PRESSURE IN THE DRYWELL BETWEEN 12:30 P.M. AND 2:30 P.M. ON DECEMBER 8, 1979, INDICATED THAT PRIMARY CONTAINMENT INTEGRITY WAS NOT BEING MAINTAINED. THE CAUSE OF LEAKAGE WAS NOT KNOWN AT THAT TIME AND CONTAINMENT INTEGRITY WAS NOT RESTORED UNTIL ABOUT 8:30 A.M. ON DECEMBER 9, 1979. REACTOR OPERATION WAS CONTINUED THROUGHOUT THIS TIME.
(7945 VIOLATION)

CONTRARY TO TECHNICAL SPECIFICATION 6.7.2.A, VERBAL NOTIFICATION TO THE NRC OF AN EVENT IN WHICH ABNORMAL DEGRADATION WAS DISCOVERED IN THE PRIMARY CONTAINMENT WAS NOT MADE UNTIL APPROXIMATELY 41 HOURS AFTER INDICATIONS WERE AVAILABLE THAT THE PRIMARY CONTAINMENT WAS LEAKING IN EXCESS OF TECHNICAL SPECIFICATION LIMITS. THE CONFIRMATION FACSIMILE TRANSMISSION WAS NOT MADE UNTIL THE SECOND WORKING DAY FOLLOWING THE EVENT.
(7945 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

+ 99% FULL POWER OPERATIONS.

+ LAST IE SITE INSPECTION DATE: FEBRUARY 26-29, 1980

+ INSPECTION REPORT NO: 50-296/80-10

SUMMARY

INSPECTION SUMMARY

+ INSPECTION DECEMBER 17-20 (79-44): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 10 INSPECTOR-HOURS ON SITE IN THE AREAS OF CONFIRMATORY MEASUREMENTS INCLUDING: REVIEW OF RADIOCHEMICAL INSTRUMENT CALIBRATION AND QUALITY CONTROL; REVIEW OF RADIOCHEMICAL PROCEDURES; AND COMPARISON OF RESULTS OF THE ANALYSES OF SPLIT SAMPLES ANALYZED BY THE LICENSEE AND THE NRC RII MOBILE LABORATORY. OF THE THREE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN TWO AREAS; ONE ITEM OF NONCOMPLIANCE WAS FOUND IN ONE AREA (DEFICIENCY - FAILURE TO PERFORM ANNUAL EFFICIENCY AND ACTIVITY CHECKS FOR GE(LI) COUNTING SYSTEMS).

INSPECTION JANUARY 16-18 (80-01): THIS UNANNOUNCED INSPECTION INVOLVED 16 INSPECTOR-HOURS ON SITE IN THE AREAS OF ADMINISTRATIVE CONTROLS, THERMAL AND CHEMICAL DISCHARGES, CHEMICAL USAGE, LICENSEE EVENT REPORTS AND ENVIRONMENTAL SURVEILLANCE. OF THE FIVE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
80-001/03L-0	12/30/79	01/25/80	30 DAY LER	FEEDER CABLE FAILURE BUS TIE BOARD.
80-002/03L-0	01/17/80	02/08/80	30 DAY LER	RESIDUAL HEAT REMOVAL FLOW VALVE INOPERABLE.

INSPECTION JANUARY 21-24 (80-03): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 11 INSPECTOR-HOURS ON SITE IN THE AREAS OF OBSERVATION OF THE REFUELING ACTIVITIES OF UNIT 1 IN PREPARATION FOR CYCLE 4 AND REVIEW OF THE STARTUP TESTING ACTIVITIES CONDUCTED ON UNIT 3 AT THE START OF CYCLE 3. OF THE TWO AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION JANUARY 21 (80-04): MANAGEMENT MEETING TO DISCUSS THE IMPLEMENTATION OF REQUIREMENTS DIRECTED IN THE ORDER MODIFYING THE LICENSES OF THE BROWNS FERRY NUCLEAR PLANT, UNIT NOS. 1, 2 AND 3. THE MEETING INVOLVED 4 INSPECTOR-HOURS BY 6 NRC INSPECTORS. WITHIN THE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

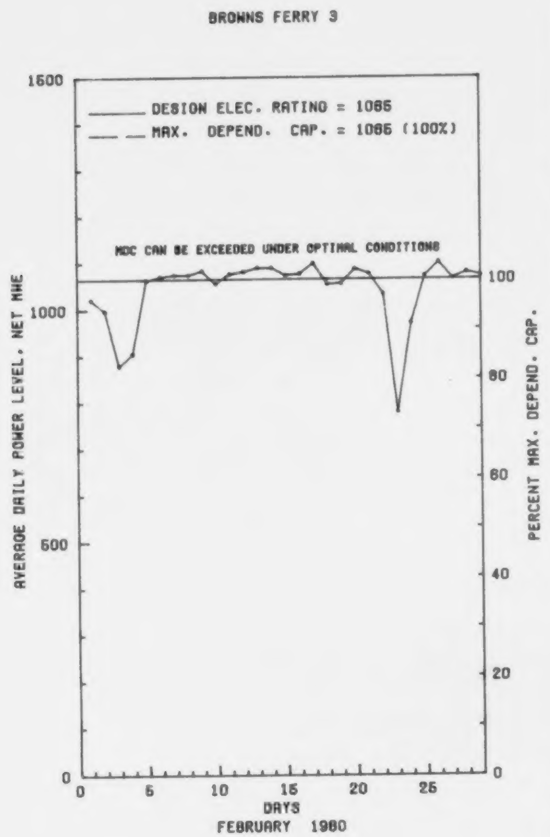
INSPECTION FEBRUARY 5-7 (80-07): THIS SPECIAL, ANNOUNCED INSPECTION INVOLVED 5 INSPECTOR-HOURS ON SITE IN THE AREAS OF VIDEO (TV) OBSERVATION OF CORE SPRAY SYSTEM SPARGER (UNIT 1); REVIEW OF METHODS OF WELD FITUP INSPECTION (UNITS 1, 2 AND 3); AND REVIEW OF PROPOSED ACTIONS CONCERNING A LICENSEE EVENT REPORT (UNIT 1). OF THE THREE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION JANUARY 29-31 (80-09): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 4 INSPECTOR-HOURS ON SITE IN THE AREA OF REFUELING ACTIVITIES. OF THE ONE AREA INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 4-10-80
REVIEWED BY NRR DOR: Dick Clark **DATE:** 4-11-80

BROWNS FERRY 3

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-296
2. Utility Contact: Don Green (205) 729-6846
3. Licensed Thermal Power (MWt): 3293
4. Nameplate Rating (Gross MWe): 1280 x 0.9 = 1152
5. Design Electrical Rating (Net MWe): 1065
6. Maximum Dependable Capacity (Gross MWe): 1098
7. Maximum Dependable Capacity (Net MWe): 1065
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	696.0	1,440.0	26,304.0
12. Hours Reactor Was Critical	696.0	1,028.4	19,936.1
13. Reactor Reserve Shutdown Hours	.0	.0	1,658.4
14. Hours Generator On-Line	696.0	1,004.9	19,442.7
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	2,199,042	3,112,700	56,224,166
17. Gross Electrical Energy (MWH)	743,960	1,069,920	18,452,500
18. Net Electrical Energy (MWH)	725,906	1,020,180	17,907,961
19. Unit Service Factor	100.0	69.8	73.9
20. Unit Availability Factor	100.0	69.8	73.9
21. Unit Capacity Factor (MDC Net)	97.9	66.5	63.5
22. Unit Capacity Factor (DER Net)	97.3	66.3	63.3
23. Unit Forced Outage Rate	.0	30.2	10.4

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type^1	Duration (Hours)	Reason^2	Method of Shutting Down Reactor^3	Licensee Event Report No.	System Code^4	Component Code^5	Cause & Corrective Action to Prevent Recurrence
116	02/01/80	F	.0	H	5				DERATED DUE TO EXCESSIVE FISH IMPINGEMENT ON CCW PUMPS TRAVELING WATER SCREENS.
117	02/22/80	S	.0	H	5				DERATED FOR CONTROL ROD SEQUENCE EXCHANGE FROM "A" TO "B".

SUMMARY

OPERATION WAS AT NEAR FULL POWER DURING THE REPORT PERIOD, WITH LOAD REDUCTIONS ON 02/01/80 DUE TO FISH IMPINGEMENT ON THE COMPONENT COOLING WATER PUMP TRAVELING SCREENS, AND ON 02/22/80 FOR CONTROL ROD SEQUENCE EXCHANGE.

LEGEND

^{1}Type:
F--Forced
S--Scheduled

^{2}Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

^{3}Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

^{4}Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

^{5}Exhibit H:
Same source.

BRUNSWICK 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	North Carolina/Brunswick 3 miles N of Southport
2.	Electric Reliability Council	Southeastern
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2436
5.	Design Electrical Rating (MWE Net)	821
6.	Date of Initial Criticality	October 8, 1976
7.	Date of Commercial Operation	March 18, 1977
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Cape Fear River

Utility & Contractor Information

10.	Licensee	Carolina Power and Light Company
11.	Corporate Address	411 Fayetteville Street Raleigh, North Carolina 27602
12.	Corporate Contact	J. A. Jones Executive Vice President
13.	Architect/Engineer	United Engineers & Constructors, Inc.
14.	Nuclear Steam Supply System	GE
15.	Constructor	Brown and Root, Inc.

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R.C. Lewis
18.	IE Senior Inspector	P.J. Kellogg
19.	IE Resident Inspector	J. E. Outzs (919) 457-9531
20.	Licensing Project Manager	J. Hannon
21.	Docket Number	50-325
22.	License Number/Date of Issue	DPR-71, 9/8/76
23.	Public Document Room	Southport-Brunswick County Library 109 W. Moore Street Southport, North Carolina 28461

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO 10 CFR 20.203(F), WHICH REQUIRES THAT EACH CONTAINER OF LICENSED MATERIAL CONTAINING QUANTITIES GREATER THAN THOSE LISTED IN APPENDIX C OF 10 CFR 20 SHALL BEAR A DURABLE CLEARLY VISIBLE LABEL IDENTIFYING THE RADIOACTIVE CONTENTS AND SHALL BEAR THE RADIATION CAUTION SYMBOL AND THE WORDS "CAUTION, RADIOACTIVE MATERIAL" OR "DANGER RADIOACTIVE MATERIAL," ON JANUARY 24, 1980, THE LICENSEE HAD NOT LABELED WITH THE APPROPRIATE WARNING AND INFORMATION SEVERAL CONTAINERS OF RADIOACTIVE MATERIAL STORED IN THE GENERAL AREA BETWEEN THE PLANT AND THE DIESEL GENERATOR BUILDING. RADIATION LEVELS MEASURED BY THE INSPECTOR INDICATED THAT THE CONTAINERS CONTAINED QUANTITIES OF RADIOACTIVE MATERIAL IN EXCESS OF THE QUANTITIES SPECIFIED IN 10 CFR 20 APPENDIX C BELOW WHICH LABELING IS NOT REQUIRED. THE EXCEPTIONS TO LABELING REQUIREMENTS SPECIFIED BY 10 CFR 20.203(F)(3) DID NOT APPLY. (8003 INFRACTION)

CONTRARY TO 10 CFR 20.201(B) WHICH REQUIRES THAT EACH LICENSEE MAKE OR CAUSE TO BE MADE SUCH SURVEYS AS MAY BE NECESSARY TO COMPLY WITH THE REGULATIONS IN 10 CFR 20, ON JANUARY 25, 1980, THE LICENSEE HAD NOT PERFORMED THE SURVEYS NECESSARY TO PREVENT LOSS OF CONTROL OVER TWO CONTAMINATED FIRE HOSES FOUND NEAR THE BASE OF THE PLANT STACK. THESE FIRE HOSES WERE NOT IN ANY TYPE OF CONTAINER AND CARRIED NO LABELS YET DOSE RATES OF UP TO 40 MR/HR WERE MEASURED AT CONTACT WITH THE HOSES. (8003 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

UNIT IN NORMAL POWER OPERATION.

+ LAST IE SITE INSPECTION DATE: JANUARY 23-25, 1980

+ INSPECTION REPORT NO: 50-325/80-03

SUMMARY

INSPECTION SUMMARY

+ INSPECTION JANUARY 22-23 (80-02): THIS SPECIAL, ANNOUNCED INSPECTION INVOLVED 13 INSPECTOR-HOURS AT UNITED ENGINEERS AND CONSTRUCTORS OFFICES IN THE AREAS OF SEISMIC ANALYSIS FOR AS-BUILT SAFETY RELATED PIPING SYSTEMS (IE BULLETIN 79-14) AND OF CONCRETE EXPANSION ANCHOR INSTALLATION (IE BULLETIN 79-02). OF THE TWO AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION JANUARY 23-25 (80-03): THIS SPECIAL, UNANNOUNCED INSPECTION INVOLVED 11 INSPECTOR-HOURS ON SITE IN THE AREAS OF PACKAGING LOW-LEVEL RADIOACTIVE WASTE FOR TRANSPORT AND BURIAL (IE BULLETIN NO. 79-19) AND ONSITE RADIOACTIVE WASTE STORAGE. OF THE TWO AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN ONE AREA; TWO APPARENT ITEMS OF NONCOMPLIANCE WAS FOUND IN ONE AREA (INFRACTION - FAILURE TO LABEL CONTAINERS OF RADIOACTIVE MATERIAL; AND INFRACTION - FAILURE TO SURVEY).

REPORTS RECEIVED FROM LICENSEE

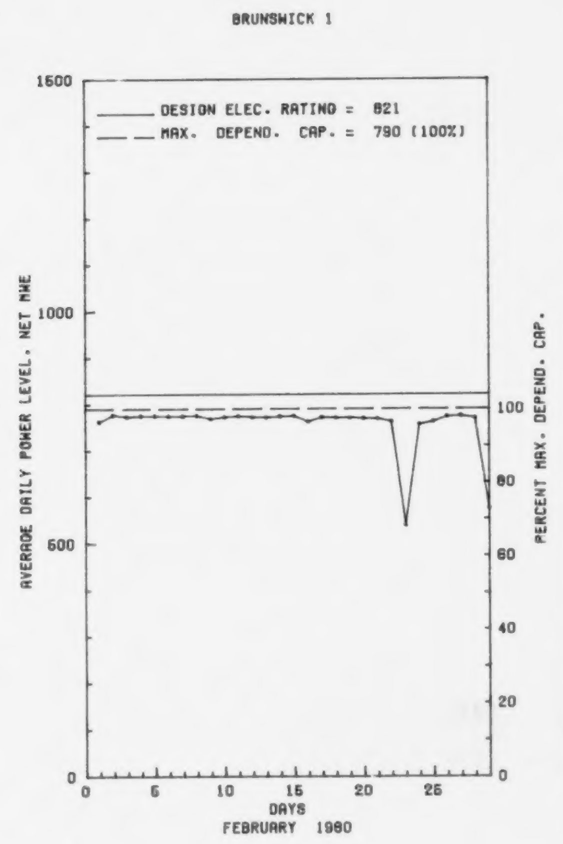
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-100/ 03L-0	12/28/79	01/23/80	30 DAY LER	THREE SNUBBERS FOUND INOPERABLE DURING VISUAL INSPECTION.
79-109/ 03L-0	12/29/79	01/23/80	30 DAY LER	ROD DRIFT ALARM ON 02-31 DUE TO FAULTY REED SWITCH.
79-110/ 03L-0	12/18/79	01/15/80	30 DAY LER	REACTOR VESSEL SHROUD LEVEL SWITCH NO. 1 ON 1-B21-LITS-N037 WAS OUT OF CALIBRATION.
79-111/ 03L-0	12/25/79	01/18/80	30 DAY LER	CONTAINMENT ATMOSPHERE MONITORS CAC-AQH-1261 AND 1262 CHANNELS 2 AND 3 FAILED DOWNSCALE.
79-113/ 03L-0	12/19/79	01/14/80	30 DAY LER	RHR TORUS SUCTION VALVE F004C FOUND SHUT DURING NORMAL REACTOR STARTUP WHILE IN CONDITION.
79-114/ 03L-0	12/27/79	01/18/80	30 DAY LER	MAIN STEAM LINE HIGH RADIATION DRAWER 1-D12-K603B OUT OF CALIBRATION I.E.3.9XBACKGROUND VERSUS 3.5XBACKGROUND.
79-115/ 03L-0	12/24/79	01/17/80	30 DAY LER	CONTAINMENT ATMOSPHERIC MONITOR LAC-AR-1263 FAILED DOWNSCALE.
79-116/ 03L-0	12/22/79	01/16/80	30 DAY LER	TORUS LEVEL INSTRUMENT CAC -LT-2602 NOT OPERATING I.E.-29 INCHES VS ACTUAL LEVEL OF -24.5.
79-117/ 03L-0	12/22/79	01/17/80	30 DAY LER	CONTAINMENT ATMOSHPERIC MONITOR CAC-AT-1263 OUT OF CALIBRATION DUE TO ZERO SHIFT.
79-118/ 03L-0	12/28/79	01/23/80	30 DAY LER	TWO SNUBBERS FOUND INOPERABLE DURING VISUAL INSPECTION.
80-01/ 03L-0	01/02/80	01/22/80	30 DAY LER	CONTAINMENT ATMOSPHERIC MONITOR CAC-AT-1263 OUT OF CALIBRATION.
80-02/ 03L-0	01/06/80	01/30/80	30 DAY LER	DEFECTIVE RELAY IN CHLORIDE INTRUSION MONITOR 1-CO-CR24.
80-05/ 03L-0	01/13/80	01/29/80	30 DAY LER	CONTROL ROD NO. 42-35 HAD NO POSITION INDICATION AT NOTCH 47 BELIEVED TO BE DUE TO FAULTY REED SWITCH.

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 4-10-80

REVIEWED BY NRR DOR: John N. Hannon **DATE:** 4-14-80

BRUNSWICK 1

AVERAGE POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-325
2. Utility Contact: F. Willis (919) 457-9521
3. Licensed Thermal Power (MWt): 2436
4. Nameplate Rating (Gross MWe): 963 x 0.9 = 867
5. Design Electrical Rating (Net MWe): 821
6. Maximum Dependable Capacity (Gross MWe): 815
7. Maximum Dependable Capacity (Net MWe): 790
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	696.0	1,440.0	25,897.0
12. Hours Reactor Was Critical	696.0	1,440.0	18,829.6
13. Reactor Reserve Shutdown Hours	.0	.0	1,647.1
14. Hours Generator On-Line	696.0	1,440.0	17,772.8
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,625,306	3,272,725	37,101,721
17. Gross Electrical Energy (MWH)	543,925	1,095,227	12,336,364
18. Net Electrical Energy (MWH)	526,332	1,059,022	11,866,832
19. Unit Service Factor	100.0	100.0	68.6
20. Unit Availability Factor	100.0	100.0	68.6
21. Unit Capacity Factor (MDC Net)	95.7	93.1	58.0
22. Unit Capacity Factor (DER Net)	92.1	89.6	55.8
23. Unit Forced Outage Rate	.0	.0	20.1

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUELING - 04/28/80
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
003	02/23/80	S	4.6	F	5		RC	ZZZZZZ	REDUCED POWER FOR A CONTROL ROD SEQUENCE EXCHANGE. A SEQUENCE EXCHANGE IS NECESSARY FOR UNIFORM FUEL BURN THROUGHOUT THE FUEL CYCLE. PLANT PROCEDURES REQUIRE A POWER REDUCTION IN ORDER TO MAKE ROD PULLS.

SUMMARY

OPERATION WAS ROUTINE DURING THE REPORT PERIOD, WITH A LOAD REDUCTION ON 02/23/80 FOR CONTROL ROD SEQUENCE EXCHANGE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

BRUNSWICK 2

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	North Carolina/Brunswick 3 miles N of Southport
2.	Electric Reliability Council	Southeastern
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2436
5.	Design Electrical Rating (MWE Net)	821
6.	Date of Initial Criticality	March 20, 1975
7.	Date of Commercial Operation	November 3, 1975
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Cape Fear River

Utility & Contractor Information

10.	Licensee	Carolina Power and Light Company
11.	Corporate Address	411 Fayetteville Street Raleigh, North Carolina 27602
12.	Corporate Contact	J. A. Jones Executive Vice President
13.	Architect/Engineer	United Engineers & Constructors, Inc.
14.	Nuclear Steam Supply System	GE
15.	Constructor	Brown and Root, Inc.

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R.C. Lewis
18.	IE Senior Inspector	P.J. Kellogg
19.	IE Resident Inspector	J. E. Outzs (919) 457-9531
20.	Licensing Project Manager	J. Hannon
21.	Docket Number	50-324
22.	License Number/Date of Issue	DPR-62, 12/27/74
23.	Public Document Room	Southport-Brunswick County Library 109 W. Moore Street Southport, North Carolina 28461

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO 10 CFR 20.203(F), WHICH REQUIRES THAT EACH CONTAINER OF LICENSED MATERIAL CONTAINING QUANTITIES GREATER THAN THOSE LISTED IN APPENDIX C OF 10 CFR 20 SHALL BEAR A DURABLE CLEARLY VISIBLE LABEL IDENTIFYING THE RADIOACTIVE CONTENTS AND SHALL BEAR THE RADIATION CAUTION SYMBOL AND THE WORDS "CAUTION, RADIOACTIVE MATERIAL" OR "DANGER RADIOACTIVE MATERIAL." ON JANUARY 24, 1980, THE LICENSEE HAD NOT LABELED WITH THE APPROPRIATE WARNING AND INFORMATION SEVERAL CONTAINERS OF RADIOACTIVE MATERIAL STORED IN THE GENERAL AREA BETWEEN THE PLANT AND THE DIESEL GENERATOR BUILDING. RADIATION LEVELS MEASURED BY THE INSPECTOR INDICATED THAT THE CONTAINERS CONTAINED QUANTITIES OF RADIOACTIVE MATERIAL IN EXCESS OF THE QUANTITIES SPECIFIED IN 10 CFR 20 APPENDIX C BELOW WHICH LABELING IS NOT REQUIRED. THE EXCEPTIONS TO LABELING REQUIREMENTS SPECIFIED BY 10 CFR 20.203(F)(3) DID NOT APPLY. (8003 INFRACTION)

CONTRARY TO 10 CFR 20.201(B) WHICH REQUIRES THAT EACH LICENSEE MAKE OR CAUSE TO BE MADE SUCH SURVEYS AS MAY BE NECESSARY TO COMPLY WITH THE REGULATIONS IN 10 CFR 20. ON JANUARY 25, 1980, THE LICENSEE HAD NOT PERFORMED THE SURVEYS NECESSARY TO PREVENT LOSS OF CONTROL OVER TWO CONTAMINATED FIRE HOSES FOUND NEAR THE BASE OF THE PLANT STACK. THESE FIRE HOSES WERE NOT IN ANY TYPE OF CONTAINER AND CARRIED NO LABELS YET DOSE RATES OF UP TO 40 MR/HR WERE MEASURED AT CONTACT WITH THE HOSES. (8003 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

* UNIT SHUTDOWN FEBRUARY 29, 1980 FOR REFUELING AND TORUS MODIFICATION.

* LAST IE SITE INSPECTION DATE: JANUARY 23-25, 1980

* INSPECTION REPORT NO: 50-324/80-03

SUMMARY

INSPECTION SUMMARY

* INSPECTION JANUARY 22-23 (80-02): THIS SPECIAL, ANNOUNCED INSPECTION INVOLVED 13 INSPECTOR-HOURS AT UNITED ENGINEERS AND CONSTRUCTORS OFFICES IN THE AREAS OF SEISMIC ANALYSIS FOR AS-BUILT SAFETY RELATED PIPING SYSTEMS (IE BULLETIN 79-14) AND OF CONCRETE EXPANSION ANCHOR INSTALLATION (IE BULLETIN 79-02). OF THE TWO AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION JANUARY 23-25 (80-03): THIS SPECIAL, UNANNOUNCED INSPECTION INVOLVED 11 INSPECTOR-HOURS ON SITE IN THE AREAS OF PACKAGING LOW-LEVEL RADIOACTIVE WASTE FOR TRANSPORT AND BURIAL (IE BULLETIN NO. 79-19) AND ONSITE RADIOACTIVE WASTE STORAGE. OF THE TWO AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN ONE AREA; TWO APPARENT ITEMS OF NONCOMPLIANCE WERE FOUND IN ONE AREA (INFRACTION - FAILURE TO LABEL CONTAINERS OF RADIOACTIVE MATERIAL; AND INFRACTION - FAILURE TO SURVEY).

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

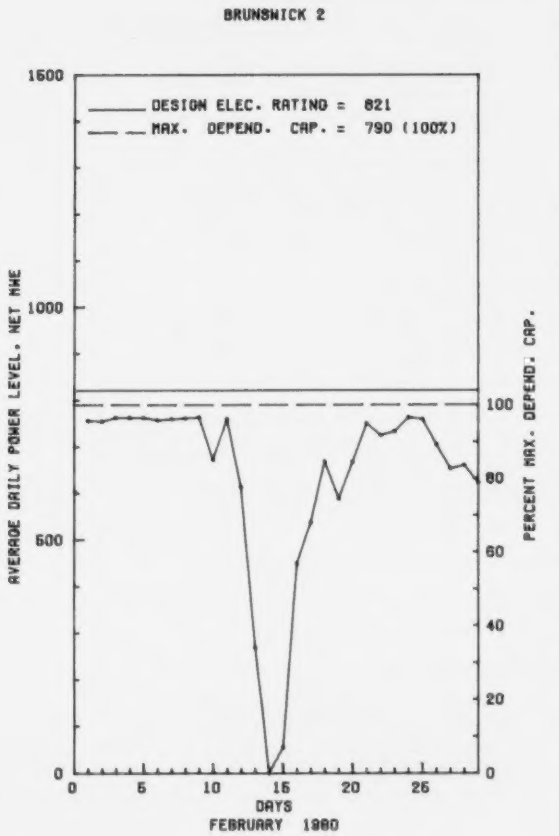
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-80/ 03L-0	01/10/80	02/06/80	30 DAY LER	RCIC TRIP AND THROTTLE VALVE MOV-V8 WOULD NOT TRIP WHEN GIVEN AN ISOLATION SIGNAL.
79-97/ 03L-0	12/17/79	01/15/80	30 DAY LER	TORUS TO DRYWELL VACUUM BREAKER LAC-X18F WITH NO CLOSED INDICATION.
79-104/ 03L-0	12/26/79	01/18/80	30 DAY LER	MAIN STEAM HIGH RADIATION DRAWER 2-012-K603D WAS OUT OF CALIBRATION I.E. 3.4 VS 4.4 X BACKGROUND.
80-02/ 03L-0	01/02/80	01/22/80	30 DAY LER	FULL-IN REED SWITCH FOR ROD 34-27 DEFECTIVE.
80-03/ 03L-0	01/03/80	01/29/80	30 DAY LER	REACTOR COOLANT EXCEEDED 2 MICRO MHOS FOR 31 HOURS.
80-05/ 03L-0	01/05/80	01/30/80	30 DAY LER	ISOLATION VALVE FOR PRESSURE SWITCH 2-E11-PS-N010D WAS FOUND CLOSED DURING NORMAL OPERATION.
80-06/ 03L-0	01/03/80	01/29/80	30 DAY LER	NO INDICATION ON CONTROL RODS NO. 26-07 AT NOTCH 48 AND ON 34-35 AT NOTCH 7 DUE TO SUSPECTED FAULTY REED SWITCHES.
80-09/ 03L-0	01/14/80	01/29/80	30 DAY LER	CONTAINMENT ATMOSPHERE MONITOR CAC-AT-1259-1 FAILED UPSCALE DUE TO LOOSE GROUND WIRE.
80-11/ 03L-0	01/10/80	02/01/80	30 DAY LER	COMPUTOR FAILURE PLUS TEMPERATURE RECORDER FAILURE CAUSED LCO ON CAC-TR-1258 POINT 14.
80-17/ 01T-0	01/18/80	01/21/80	14 DAY LER	OPERATION WITH AN IDENTIFIED INOPERABLE SNUBBER.

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 4-10-80

REVIEWED BY NRR DOR: John M. Hannon **DATE:** 4-14-80

BRUNSWICK 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-324
2. Utility Contact: F. Willis (919) 457-9521
3. Licensed Thermal Power (MWt): 2436
4. Nameplate Rating (Gross MWe): 963 x 0.9 = 867
5. Design Electrical Rating (Net MWe): 821
6. Maximum Dependable Capacity (Gross MWe): 815
7. Maximum Dependablo Capacity (Net MWe): 790
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	696.0	1,440.0	37,921.0
12. Hours Reactor Was Critical	660.0	1,363.0	26,686.9
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	639.6	1,225.4	25,099.4
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,370,499	2,600,489	46,336,951
17. Gross Electrical Energy (MWH)	459,420	871,110	15,486,184
18. Net Electrical Energy (MWH)	444,467	842,126	14,862,598
19. Unit Service Factor	91.9	85.1	66.2
20. Unit Availability Factor	91.9	85.1	66.2
21. Unit Capacity Factor (MDC Net)	80.8	74.0	49.7
22. Unit Capacity Factor (DER Net)	77.8	71.2	47.8
23. Unit Forced Outage Rate	8.1	8.1	12.8

24. Shutdowns Scheduled Over Next 6 Months (Type, Data, and Duration of Each): REFUELING - 03/01/80
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
003	02/12/80	F	9.1	A	5		HC	HTEXCH	REDUCED REACTOR POWER TO PERFORM CONDENSER WATERBOX WORK. 2A-SOUTH CONDENSER WAS FOUND WITH 4 LEAKING TUBES. THESE TUBES WERE PLUGGED.
004	02/13/80	F	56.5	B	3		IA	INSTRU	REACTOR SCRAM DURING PERFORMANCE OF I&C PERIODIC TEST 3.1.14 PC. REACTOR LOW PRESSURE CHANNEL CALIBRATION AND FUNCTIONAL TEST. A SLIGHT MECHANICAL JAR OF THE ISOLATION VALVE FOR THE INSTRUMENT BEING TESTED, APPARENTLY TRANSMITTED TO ANOTHER INSTRUMENT, PRODUCED THE REACTOR SCRAM.
005	02/19/80	F	8.1	A	5		AA	BLOWER	A POWER REDUCTION BECAUSE OF RECIRCULATION PUMP RUNBACK CAUSED BY A PROBLEM WITH REACTOR FEED PUMP A. THE RELAY WHICH SENSES LOSS OF CONTROL SIGNAL HAD DEENERGIZED, CAUSING THE REACTOR FEED PUMP A TO RUN BACK TO MINIMUM SPEED.

SUMMARY

OPERATION WAS ROUTINE DURING THE REPORT PERIOD, WITH LOAD REDUCTIONS ON 02/12/80 FOR CONDENSER WATERBOX MAINTENANCE AND ON 02/19/80 BECAUSE OF A REACTOR RECIRCULATION PUMP RUNBACK. A REACTOR SCRAM OCCURRED ON 02/13/80 BECAUSE OF A MECHANICAL JAR TO AN INSTRUMENT DURING PERIODIC TESTING.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

CALVERT CLIFFS 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Maryland/Calvert 40 miles S of Annapolis
2.	Electric Reliability Council	MAAC
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2700
5.	Design Electrical Rating (MWE Net)	845
6.	Date of Initial Criticality	October 7, 1974
7.	Date of Commercial Operation	May 8, 1975
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Chesapeake Bay

Utility & Contractor Information

10.	Licensee	Baltimore Gas and Electric Company
11.	Corporate Address	Gas and Electric Building Baltimore, Maryland 21203
12.	Corporate Contact	A. E. Lundvall, Jr. Vice President-Supply
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Combustion Engineering
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Senior Inspector	R. Keimig
19.	IE Resident Inspector	R. Architzel (301) 586-2626
20.	Licensing Project Manager	E. L. Conner, Jr.
21.	Docket Number	50-317
22.	License Number/Date of Issue	DPR-53, 7/31/74
23.	Public Document Room	Calvert County Library Prince Frederick, Maryland 20678

INSPECTION STATUS

ENFORCEMENT STATUS

SECTION 2.2.1 OF APPENDIX B, ENVIRONMENTAL TECHNICAL SPECIFICATIONS, REQUIRES IN PART, THAT THE DEMINERALIZER REGENERATION PRODUCED WASTE SOLUTION SHALL BE NEUTRALIZED, IF NECESSARY, TO OBTAIN A PH IN THE RANGE OF 6.0-9.0 PRIOR TO RELEASE INTO THE CIRCULATING WATER DISCHARGE. CONTRARY TO THE ABOVE, ON NOVEMBER 29, 1979, DEMINERALIZER REGENERATION PRODUCED WASTE SOLUTION WITH A PH OF 12.1, WAS NOT NEUTRALIZED TO OBTAIN A PH IN THE RANGE OF 6.0 TO 9.0 PRIOR TO RELEASE, AND WAS RELEASED TO THE YARD DRAIN INSTEAD OF TO THE CIRCULATING WATER DISCHARGE, AS SPECIFIED.
(7924 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS:

+ No significant information to report.

+ LAST IE SITE INSPECTION DATE: 3/12/80

+ INSPECTION REPORT NO: 50-317/80-03

SUMMARY

INSPECTION SUMMARY

+ 50-317/79-24 - Dec 10-11: Special, announced inspection of the make-up and condensate demineralizer system operations and associated nonradiological discharges. The inspection involved 14 hours onsite by two regionally based NRC inspectors. Subsequent to the inspection an enforcement meeting was held on November 29, 1979 at the NRC Region I office. Noncompliances: One (exceeding the plant discharge limiting condition for operation for pH).

REVIEWED BY OMPA LOEB: Mk Beebe **DATE:** 04/10/80

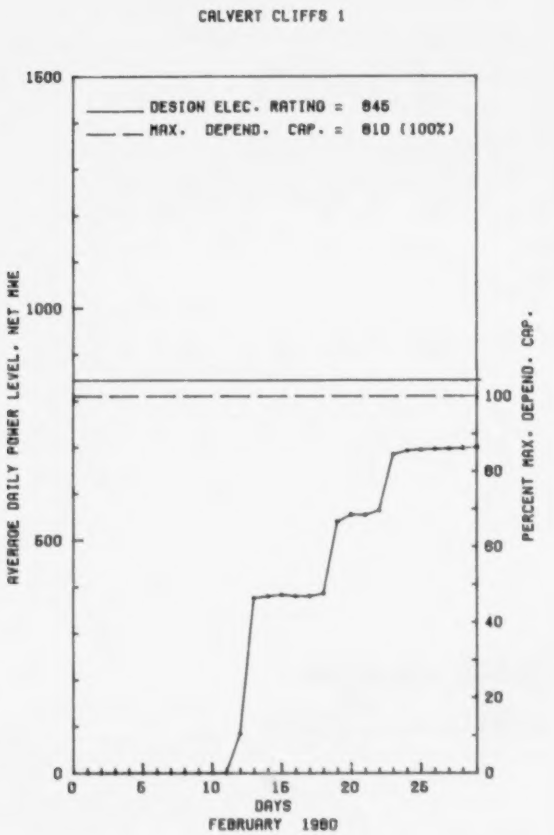
REVIEWED BY NRR DOR: ________ **DATE:** ________

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-052/ 03L	10/19/79	11/06/79	30 Day LER	Failure of the containment gaseous and particulate radioactivity monitoring system due to loss of the sampling pump
79-053/ 03L	09/28/79	10/25/79	30 Day LER	As a result of a broken actuator arm #11 shutdown cooling heat exchanger outlet valve was inoperable rendering #11 shutdown cooling heat exchanger inoperable
79-054/ 03L	02/29/79	10/25/79	30 Day LER	During operation failure of a welded cap on #11 component cooling heat exchanger salt water outlet valve rendered the #11 salt water system inoperable
79-055/ 03L	09/30/79	11/02/79	30 Day LER	During power operation a module failure caused the channel A power distribution setpoint to decrease. The channel was placed in bypass and subsequently repaired
79-058/ 03L	10/04/79	11/08/79	30 Day LER	Due to a faulty pneumatic converter #11 salt water outlet valve would not operate properly
79-059/ 03L	10/26/79	11/13/79	30 Day LER	During routine surveillance two fire hoses were found damaged. Both hoses were replaced
79-060/ 03L	10/20/79	11/15/79	30 Day LER	During a load test on #12 diesel generator high exhaust temperatures were experienced which resulted in the inability to achieve full load capability of 2500 KW
79-061/ 01T	10/24/79	11/07/79	Special	As a result of followup on IEB 79-14 it was discovered that the air start systems for the diesel generators did not meet seismic requirements
79-063/ 04T	10/22/79	11/05/79	14 Day LER	Oyster Samples collected on August 27, 1979 showed the presence of AG-110M at 266 plus or minus 4% p-ci/kg
79-064/ 03L	11/01/79	11/13/79	30 Day LER	Fire barriers between the service building and intake structure were temporarily removed to facilitate cable installation
79-065/ 03L	11/13/79	12/04/79	30 Day LER	Emergency diesel generator removed from service to repair woodward governor
79-066/ 03L	11/20/79	12/10/79	30 Day LER	Containment cooler fan inoperable (hand switch failure)
79-068/ 03L	11/27/79	12/13/79	30 Day LER	No. 12 emergency diesel inoperable to repair service water vent valve
79-079/ 03L	12/19/79	01/10/80	30 Day LER	Both spent fuel pool exhaust fans were taken out of service to replace roughing filters
79-081/ 03L	12/25/79	01/25/80	30 Day LER	Containment air particulate and gaseous radiation monitor pump tripped off due to low air flow
80-001/ 03L	01/04/80	01/31/80	30 Day LER	HPSI loop stop opening only 5 percent
80-003/ 03L	01/08/80	02/07/80	30 Day LER	Containment air lock leakage exceeded allowable leakage
80-004/ 03L	01/10/80	02/08/80	30 Day LER	Containment radiation monitor OOS for 3.5 hr because of low flow caused by gasket leakage
80-006/ 03L	01/03/80	01/31/80	30 Day LER	Control rod dropped during surveillance testing
80-007/ 01T	01/29/80	02/12/80	Special	While shutdown conducting a core flush a PORV opened reducing pressure from 285 psia to 169 psia
80-008/ 03L	01/26/80	02/25/80	30 Day LER	Thru-wall leak in reactor coolant pump delta-p XMTR sensing line isolated by shutting root valve. Interim repair effected by installing new section with swagelock fittings
80-010/ 03L	02/02/80	02/08/80	30 Day LER	Emergency diesel power unavailable for 15 minutes after false SIAS signal generated on Unit 1. Diesels were secured without proper SIAS reset

CALVERT CLIFFS 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-317
2. Utility Contact: S. D. Merson (301) 234-5240
3. Licensed Thermal Power (MWt): 2700
4. Nameplate Rating (Gross MWe): 1020 x 0.9 = 918
5. Design Electrical Rating (Net MWe): 845
6. Maximum Dependable Capacity (Gross MWe): 845
7. Maximum Dependable Capacity (Net MWe): 810
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): 790
10. Reasons for Restrictions, If Any:
REDUCED TO 790 MW DUE TO TURBINE BLADE PROBLEMS

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	696.0	1,440.0	42,205.0
12. Hours Reactor Was Critical	451.8	1,052.2	33,491.8
13. Reactor Reserve Shutdown Hours	.0	.0	1,060.5
14. Hours Generator On-Line	428.0	1,028.0	32,730.0
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	701,961	1,711,638	27,049,676
17. Gross Electrical Energy (MWH)	244,542	516,670	25,329,200
18. Net Electrical Energy (MWH)	227,541	478,003	24,127,209
19. Unit Service Factor	61.5	71.4	77.6
20. Unit Availability Factor	61.5	71.4	77.6
21. Unit Capacity Factor (MDC Net)	40.4	41.0	70.6
22. Unit Capacity Factor (DER Net)	38.7	39.3	67.7
23. Unit Forced Outage Rate	.0	.0	8.7

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-01	01/25/80	S	268.0	D	4		ZZ	ZZZZZZ	TO PERFORM TMI MODIFICATIONS AS REQUIRED BY NRC.

SUMMARY

THE UNIT REMAINED SHUTDOWN UNTIL 02/12/80 FOR TMI-RELATED MODIFICATIONS. POWER WAS LIMITED TO 790 MWE (NET) FOR THE REST OF THE REPORT PERIOD DUE TO TURBINE BLADE PROBLEMS.

LEGEND

[1] Type:
ed
duled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

CALVERT CLIFFS 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Maryland/Calvert 40 miles S of Annapolis
2.	Electric Reliability Council	MAAC
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2700
5.	Design Electrical Rating (MWE Net)	846
6.	Date of Initial Criticality	November 30, 1976
7.	Date of Commercial Operation	April 1, 1977
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Chesapeake Bay

Utility & Contractor Information

10.	Licensee	Baltimore Gas and Electric Company
11.	Corporate Address	Gas and Electric Building Baltimore, Maryland 21203
12.	Corporate Contact	A. E. Lundvall, Jr. Vice President-Supply
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Combustion Engineering
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	IE Senior Inspector	R. Keimig
19.	IE Resident Inspector	R. Architzel (301) 586-2626
20.	Licensing Project Manager	E. L. Conner, Jr.
21.	Docket Number	50-318
22.	License Number/Date of Issue	DPR-69, 8/13/76
23.	Public Document Room	Calvert County Library Prince Frederick, Maryland 20678

INSPECTION STATUS

ENFORCEMENT STATUS

SECTION 2.2.1 OF APPENDIX B, ENVIRONMENTAL TECHNICAL SPECIFICATIONS, REQUIRES IN PART, THAT THE DEMINERALIZER REGENERATION PRODUCED WASTE SOLUTION SHALL BE NEUTRALIZED, IF NECESSARY, TO OBTAIN A PH IN THE RANGE OF 6.0-9.0 PRIOR 1) RELEASE INTO THE CIRCULATING WATER DISCHARGE. CONTRARY TO THE ABOVE, ON NOVEMBER 29, 1979, DEMINERALIZER REGENERATION PRODUCED WASTE SOLUTION WITH A PH OF 12.1, WAS NOT NEUTRALIZED TO OBTAIN A PH IN THE RANGE OF 6.0 TO 9.0 PRIOR TO RELEASE, AND WAS RELEASED TO THE YARD DRAIN INSTEAD OF TO THE CIRCULATING WATER DISCHARGE, AS SPECIFIED.
(7923 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS:

No significant information to report.

+ LAST IE SITE INSPECTION DATE: 3/12/80

+ INSPECTION REPORT NO: 50-318/80-03

SUMMARY

INSPECTION SUMMARY

+ 50-318/79-23 - Dec 10-11: Special, announced inspection of the make-up and condensate demineralizer system operations and associated nonradiological discharges. The inspection involved 14 hours onsite by two regionally based NRC inspectors. Subsequent to the inspection an enforcement meeting was held on November 29, 1979 at the NRC Region I office. Noncompliances: One (exceeding the plant discharge limiting condition for operation for pH).

REPORTS RECEIVED FROM LICENSEE

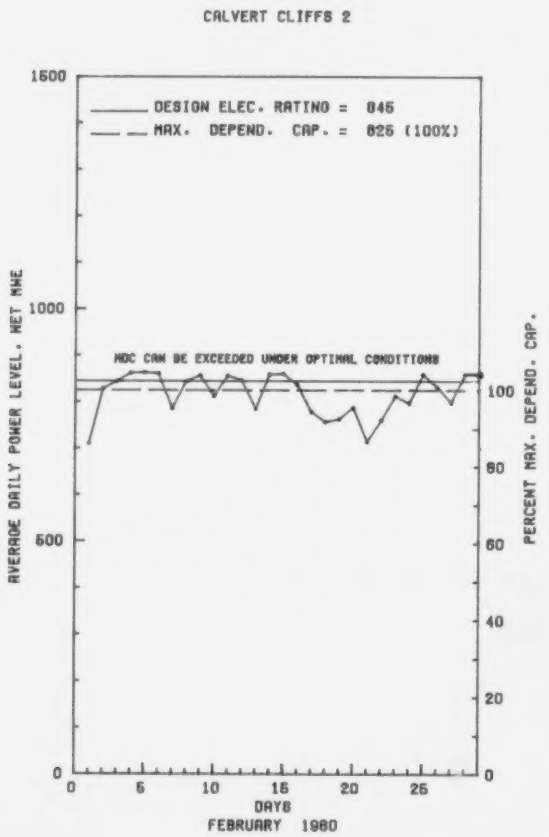
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-037/ 03L	10/05/79	11/05/79	30 Day LER	During normal operation ESFAS sensor cabinet 2F was deenergized temporarily to replace a failed voltage meter
79-050/ 03L	12/28/79	01/25/80	30 Day LER	During steady state operation channel B axial flux offset was reading higher than the other channels
79-051/ 03L	12/28/79	01/25/80	30 Day LER	Containment gaseous monitor failed
79-052/ 03L	12/26/79	01/25/80	30 Day LER	Control element assembly motion inhibit was inoperable while repairs were made to the metrascope
80-002/ 03L	01/07/80	02/07/80	30 Day LER	Refueling water tank level below TS limit by narrow range indicator whose indicator alarm led was faulty
80-003/ 0*L	01/10/80	92/08/80	30 Day LER	Diesel 21 OOS for 3.6 hr to repair jacket coolant system
80-008/ 03L	01/25/80	02/25/80	30 Day LER	Containment spray header isolated for 4.3 hr to repair shutdown HX inlet x-connect valve. The other spray header remained operable

REVIEWED BY OMPA LOEB: MR Beebe DATE: 04/10/80

REVIEWED BY NRR DOR: ________ DATE: ________

CALVERT CLIFFS 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-318
2. Utility Contact: S. D. Merson (301) 234-5240
3. Licensed Thermal Power (MWt): 2700
4. Nameplate Rating (Gross MWe): 1012 x 0.9 = 911
5. Design Electrical Rating (Net MWe): 845
6. Maximum Dependable Capacity (Gross MWe): 860
7. Maximum Dependable Capacity (Net MWe): 825
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	696.0	1,440.0	25,560.0
12. Hours Reactor Was Critical	696.0	1,220.9	21,165.8
13. Reactor Reserve Shutdown Hours	.0	.0	413.1
14. Hours Generator On-Line	696.0	1,212.7	20,821.2
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	3,772,468	5,129,644	53,373,875
17. Gross Electrical Energy (MWH)	593,761	1,047,947	17,053,170
18. Net Electrical Energy (MWH)	568,445	1,001,537	16,258,557
19. Unit Service Factor	100.0	84.2	81.5
20. Unit Availability Factor	100.0	84.2	81.5
21. Unit Capacity Factor (MDC Net)	99.0	84.3	77.1
22. Unit Capacity Factor (DER Net)	96.7	82.3	75.5
23. Unit Forced Outage Rate	.0	.0	6.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	NONE								

SUMMARY

OPERATION WAS AT NEAR FULL POWER DURING THE REPORT PERIOD, WITH SEVERAL BRIEF REDUCTIONS FOR CONDENSER MAINTENANCE.

LEGEND

[1]Type:
F--Forced
S--Scheduled

[2]Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3]Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4]Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5]Exhibit H:
Same source.

NRC POWER RESTRICTIONS: NONE

COOK 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Michigan/Berrien 11 miles S of Benton Harbor
2.	Electric Reliability Council	ECAR
3.	Type of Reactor	PWR
4.	Power Level (MWT)	3250
5.	Design Electrical Rating (MWE Net)	1054
6.	Date of Initial Criticality	January 18, 1975
7.	Date of Commercial Operation	August 27, 1975
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10.	Licensee	AEP Corporation
11.	Corporate Address	Indiana & Michigan Power Co. 2 Broadway New York, New York 10004
12.	Corporate Contact	John Dolan Senior Executive V. P. Eng.
13.	Architect/Engineer	AEP
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Indiana & Michigan Power Co.

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Senior Inspector	D.C. Boyd
19.	IE Resident Inspector	R. Masse (616) 465-6775
20.	Licensing Project Manager	D. Wigginton
21.	Docket Number	50-315
22.	License Number/Date of Issue	DPR-58 issued 10/25/74
23.	Public Document Room	Maude Preston Palenske Memorial Library 500 Market Street St. Joseph, Michigan 49085

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

Unit is in normal operation.

+LAST IE SITE INSPECTION DATE: December 17, 1979 and January 9-10, 1980

+ INSPECTION REPORT NO: 80-03

SUMMARY

INSPECTION SUMMARY

+ Inspection on December 1, 1979 through January 11, 1980 (80-02): Review of plant operations, security, Licensee Event Reports (LER's), surveillance, IE Bulletins, records, independent inspection, and License Conditions. The inspection involved 184 inspector hours onsite by the Senior Resident Inspector. Of the eight areas inspected, no items of noncompliance were identified in seven. In the independent inspection effort, one item of noncompliance was identified as follows: Control of Standing Orders.

+ Investigation on December 17, 1979 and January 9-10, 1980 (80-03): Special unannounced investigation into allegations concerning improper welding techniques and security procedures at the D. C. Cook, Unit 1, containment area; interviews of licensee and contractor personnel. The investigation involved 30 investigative-hours onsite by three NRC personnel. No items of noncompliance with NRC requirements were identified during the course of this investigation.

REPORTS RECEIVED FROM LICENSEE

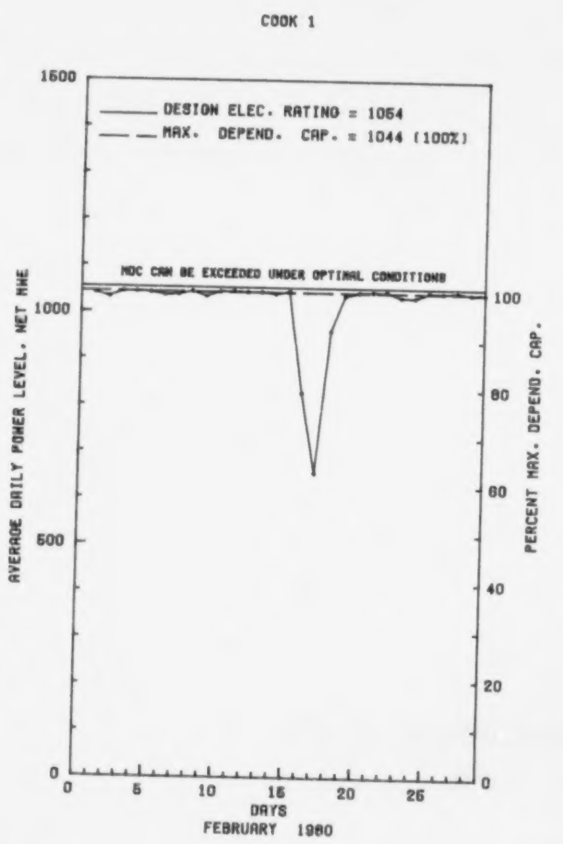
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
80-02/03L-0	01/25/80	02/22/80	30 Day LER	Loop 3T-average and Delta-T indication was decreasing and determined inoperable.

REVIEWED BY OMPA LOEB: T.C. CINTULA DATE: 4·10·80

REVIEWED BY NRR DOR: DATE: 4.11.80

COOK 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-315
2. Utility Contact: W. T. Gillett (616) 465-5901
3. Licensed Thermal Power (MWt): 3250
4. Nameplate Rating (Gross MWe): 1280 x 0.9 = 1152
5. Design Electrical Rating (Net MWe): 1054
6. Maximum Dependable Capacity (Gross MWe): 1080
7. Maximum Dependable Capacity (Net MWe): 1044
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	696.0	1,440.0	45,264.0
12. Hours Reactor Was Critical	696.0	1,042.8	34,004.5
13. Reactor Reserve Shutdown Hours	.0	.0	463.0
14. Hours Generator On-Line	696.0	1,036.6	33,119.8
15. Unit Reserve Shutdown Hours	.0	.0	321.0
16. Gross Thermal Energy (MWH)	2,209,403	3,190,053	92,256,358
17. Gross Electrical Energy (MWH)	734,460	1,057,970	30,215,310
18. Net Electrical Energy (MWH)	708,795	1,020,062	29,017,376
19. Unit Service Factor	100.0	72.0	75.0
20. Unit Availability Factor	100.0	72.0	75.0
21. Unit Capacity Factor (MDC Net)	97.5	67.9	67.0
22. Unit Capacity Factor (DER Net)	96.6	67.2	62.2
23. Unit Forced Outage Rate	.9	28.0	8.6

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
158	02/16/80	F	.0	B	5		CH	ZZZZZZ	REACTOR POWER REDUCED TO 62% TO PERMIT REMOVAL OF THE WEST MAIN FEED PUMP TURBINE FROM SERVICE TO INVESTIGATE CAUSE FOR OSCILLATION IN PUMP AND TURBINE VIBRATION. REACTOR POWER RETURNED TO 100% 02/18/80.

SUMMARY

OPERATION WAS AT NEAR FULL POWER DURING THE REPORT PERIOD. ON 02/16/80, LOAD WAS REDUCED FOR REMOVAL OF THE WEST MAIN FEED PUMP FROM SERVICE TO INVESTIGATE MINOR VIBRATION AND OSCILLATION. A BALANCE WEIGHT PLACED ON THE PUMP DID NOT HELP THE OSCILLATION; HOWEVER, THIS VIBRATION PATTERN CEASED BY 02/29/80. OTHER BRIEF REDUCTIONS WERE NECESSARY FOR TURBINE VALVE TESTING AND MINOR MAINTENANCE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

COOK 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Michigan/Berrien 11 miles S of Benton Harbor
2.	Electric Reliability Council	ECAR
3.	Type of Reactor	PWR
4.	Power Level (MWT)	3391
5.	Design Electrical Rating (MWE Net)	1100
6.	Date of Initial Criticality	March 10, 1978
7.	Date of Commercial Operation	July 1, 1978
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10.	Licensee	AEP Corporation Indiana & Michigan Power Co. Indiana & Michigan Electric Co.
11.	Corporate Address	2 Broadway New York, New York 10004
12.	Corporate Contact	John Dolan, VP/Chief Engineer (212) 422 - 4800
13.	Architect/Engineer	American Elec. Power Service Corp.
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Indiana & Michigan Power Co.

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Senior Inspector	D.C. Boyd
19.	IE Resident Inspector	R. Masse (616) 465-6775
20.	Licensing Project Manager	D. Wigginton
21.	Docket Number	50-316
22.	License Number/Date of Issue	DPR-74, 12/23/77
23.	Public Document Room	Maude Preston Palenske Memorial Library 500 Market Street St. Joseph, Michigan 49085

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

Unit in normal operations.

+LAST IE SITE INSPECTION DATE: December 1, 1979 through January 11, 1980

+ INSPECTION REPORT NO: 80-02

SUMMARY

INSPECTION SUMMARY

+ Inspections on October 31; November 1, 6, 13, 29; Decmeber 5, 6, 19, 1979, and January 7-8, 1980 (79-22): QA/QC program, implementary procedures, observation of work activities, and review and evaluation nondestructive examinations (NDE) data and reports of inservice inspection (ISI); IE Bulletin 79-13 and IE Bulletin 79-17 activities. The inspection involved 32 inspector-hours onsite and 5 hours at Southwest Research Institute by one NRC inspector. No items of noncompliance or deviations were identified.

+ Inspection on December 1, 1979 through January 11, 1980 (80-02): Review of plant operations, security, Licensee Event Reports (LER's), surveillance, IE Bulletins, records, independent inspection, and License Conditions. The inspection involved 104 inspector hours onsite by the Senior Resident Inspector. Of the eight areas inspected, no items of noncompliance were identified in seven. In the independent inspection effort, one item of noncompliance was identified as follows: Control of Standing Orders.

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
80-01/ 03L-0	01/19/80	02/06/80	30 Day LER	While in the process of performing a normal unit start-up and with the reactor power level approximately 10%, the a feedwater transient occurred.
80-02/ 03L-0	01/15/80	02/08/80	30 Day LER	During integrity testing of the south injection train, excessive leakage from the safety injection valve to the pressurizer relief tank was identified.
80-03/ 03L-0	01/24/80	02/22/80	30 Day LER	Turbine driven aux. feed pump trip and throttle valve became unlatched for no apparent reason.

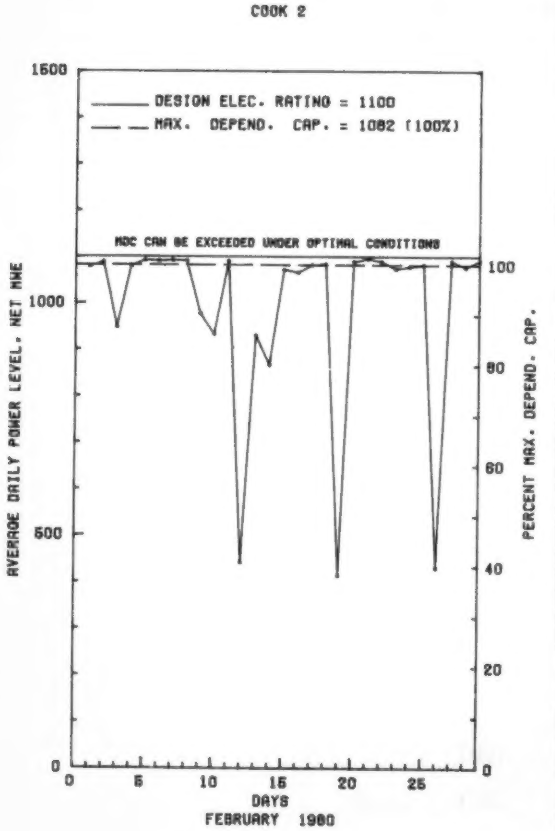
==

REVIEWED BY OMPA LOEB: T. C. CINTULA **DATE:** 4·10·80

REVIEWED BY NRR DOR: DWiggint **DATE:** 4 11 80

COOK 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-316
2. Utility Contact: W. T. Gillett (616) 465-5901
3. Licensed Thermal Power (MWt): 3391
4. Nameplate Rating (Gross MWe): 1333 x 0.85 = 1133
5. Design Electrical Rating (Net MWe): 1100
6. Maximum Dependable Capacity (Gross MWe): 1118
7. Maximum Dependable Capacity (Net MWe): 1082
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	696.0	1,440.0	18,960.0
12. Hours Reactor Was Critical	680.6	1,001.4	12,031.3
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	670.3	921.0	11,422.9
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	2,209,403	2,793,807	35,018,187
17. Gross Electrical Energy (MWH)	714,670	897,420	11,051,830
18. Net Electrical Energy (MWH)	689,185	863,137	10,630,549
19. Unit Service Factor	96.3	64.0	69.1
20. Unit Availability Factor	96.3	64.0	69.1
21. Unit Capacity Factor (MDC Net)	91.5	55.4	62.9
22. Unit Capacity Factor (DER Net)	90.0	54.5	61.8
23. Unit Forced Outage Rate	3.7	5.2	13.0

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
68	02/03/80	S	.0	B	5		ZZ	ZZZZZZ	REACTOR POWER REDUCED TO 80% TO PERFORM POWER COEFFICIENT TEST. REACTOR POWER RETURNED TO 100% 02/04/80.
69	02/10/80	S	.0	B	5		ZZ	ZZZZZZ	REACTOR POWER REDUCED TO 75% FOR TURBINE AND MOISTURE SEPARATOR REHEATER PERFORMANCE TESTING. REACTOR POWER RETURNED TO 100% THE SAME DAY.
70	02/12/80	F	10.6	A	3		HA	ZZZZZZ	TURBINE/REACTOR TRIP DUE TO EXTREME HIGH LEVEL IN LEFT MOISTURE SEPARATOR REHEATER. SPURIOUS TRIP - NO EVIDENCE OF HIGH LEVEL IN THE MOISTURE SEPARATOR. UNIT RETURNED TO SERVICE THE SAME DAY. REACTOR POWER AT 100% 02/13/80.
71	02/13/80	F	.0	H	5		ZZ	ZZZZZZ	REACTOR POWER REDUCED TO 60% DUE TO SECONDARY SIDE WATER CHEMISTRY PROBLEMS TO ENHANCE CLEANUP. REACTOR POWER RETURNED TO 100% 02/14/80.
72	02/19/80	F	8.4	H	2		HA	ZZZZZZ	UNIT TRIPPED MANUALLY DUE TO HIGH VIBRATION ON NO. 4 TURBINE BEARING. THE HIGH VIBRATION IS BELIEVED TO HAVE BEEN CAUSED BY RAPID TEMPERATURE CHANGE WHILE PLACING MOISTURE SEPARATOR REHEATER IN SERVICE.
73	02/26/80	F	6.7	G	1		ZZ	ZZZZZZ	UNIT REMOVED FROM SERVICE DUE TO HIGH VIBRATION ON TURBINE GENERATOR BEARING NO. 5. VIBRATION BELIEVED TO BE CAUSED BY RAPID TEMPERATURE CHANGE IN GENERATOR GAS TEMPERATURES CAUSED BY OPERATIONAL ERROR.

SUMMARY

BRIEF LOAD REDUCTIONS OCCURRED ON 02/03/80 AND 02/10/80 FOR TESTING, AND ON 02/13/80 DUE TO HIGH CONDUCTIVITY IN THE CONDENSATE AND FEEDWATER SYSTEM. A TURBINE/REACTOR TRIP DUE TO EXTREME HIGH LEVEL IN THE MOISTURE SEPARATOR REHEATER OCCURRED ON 02/12/80, AND THE UNIT WAS TRIPPED MANUALLY ON 02/19/80 TO INVESTIGATE HIGH VIBRATION ON THE NO. 4 TURBINE BEARING. THE UNIT WAS TAKEN OFF-LINE ON 02/26/80 DUE TO HIGH VIBRATION ON TURBINE GENERATOR BEARING NO. 5.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

COOPER STATION

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Nebraska/Nemaha 23 miles S of Nebraska City
2.	Electric Reliability Council	MARCA
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2381
5.	Design Electrical Rating (MWE Net)	778
6.	Date of Initial Criticality	February 21, 1974
7.	Date of Commercial Operation	July 1, 1974 (50% rating)
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Missouri River

Utility & Contractor Information

10.	Licensee	Nebraska Public Power District
11.	Corporate Address	P. O. Box 499 Columbus, Nebraska 68601
12.	Corporate Contact	J. M. Pilant, Director Licensing and Quality Assurance
13.	Architect/Engineer	Burns and Roe, Inc.
14.	Nuclear Steam Supply System	GE
15.	Constructor	Burns and Roe, Inc.

Regulatory Information

16.	IE Region responsible	IV
17.	IE Branch Chief	G. L. Madsen
18.	IE Senior Inspector	T. F. Westerman
19.	IE Principal Inspector	G. L. Constable
20.	Licensing Project Manager	V. L. Rooney
21.	Docket Number	50-298
22.	License Number/Date of Issue	DPR-46, 1/18/74
23.	Public Document Room	Auburn Public Library 118 15th Street Auburn, Nebraska 68305

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

```
INSPECTION STATUS

OTHER ITEMS OF INTEREST
     SYSTEMS AND COMPONENT PROBLEMS:


     FACILITY ITEMS (PLANS AND PROCEDURES):


     MANAGERIAL ITEMS:


     PLANT STATUS:
+    SHUT DOWN FOR REFULEING, TORUS MODIFICATIONS AND INSPECTION OF
     WESTINGHOUSE TURBINE FOR CRACKING ON 3/1/80.

LAST IE SITE INSPECTION DATE:  DECEMBER 3-5, 1979
INSPECTION REPORT NO:  50-298/79-17


SUMMARY
INSPECTION SUMMARY
+  NO INSPECTION CONDUCTED
```

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-03 03-L0	1/11/80	2/6/80	SUPPRESSION CHAMBER INBD. VENT. ISO. VALVE PC-230 MV NOT FULLY CLOSED.
80-04 03-L0	1/15/80	2/8/80	RHR M018 INOPERABLE IN NORMAL (CLOSED) POSITION DUE TO BLOWN FUSE.
80-05 03-L0	1/16/80	2/13/80	PIN HOLE LEAKS FOUND ON NORTH CRITICAL SUPPLY HEADER OF RECS.

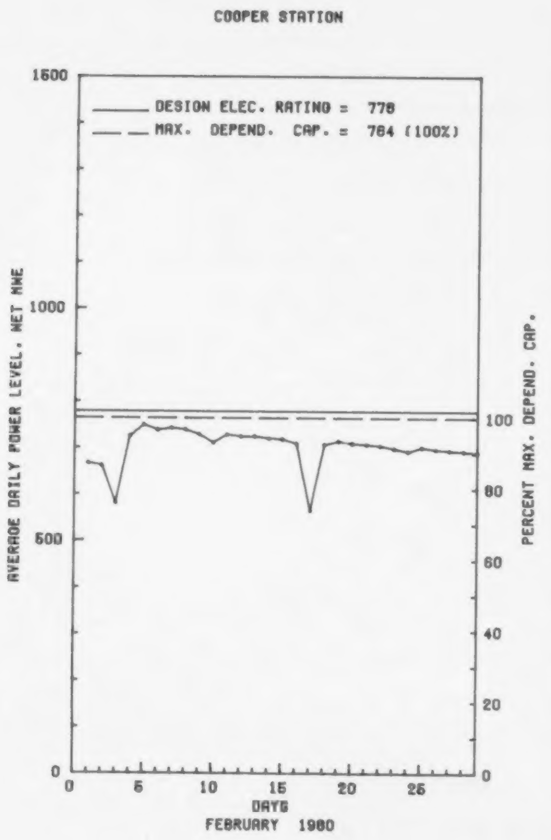
REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 4-10-80

REVIEWED BY NRR DOR: VL Rooney **DATE:** 4/11/80

GPO 871-667

COOPER STATION

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-298
2. Utility Contact: P. J. Bor r (402) 825-3811
3. Licensed Thermal Power (MWt): 2381
4. Nameplate Rating (Gross MWe): 983 x 0.85 = 836
5. Design Electrical Rating (Net MWe): 778
6. Maximum Dependable Capacity (Gross MWe): 787
7. Maximum Dependable Capacity (Net MWe): 764
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	696.0	1,440.0	49,68[illegible].0
12. Hours Reactor Was Critical	696.0	1,425.0	42,662.3
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	696.0	1,415.7	41,891.3
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,531,536	3,038,016	79,220,762
17. Gross Electrical Energy (MWH)	506,142	1,006,556	25,545,404
18. Net Electrical Energy (MWH)	487,658	969,947	24,628,224
19. Unit Service Factor	100.0	98.3	84.3
20. Unit Availability Factor	100.0	98.3	84.3
21. Unit Capacity Factor (MDC Net)	91.7	88.2	64.9
22. Unit Capacity Factor (DER Net)	90.1	86.6	63.7
23. Unit Forced Outage Rate	.0	1.7	4.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUELING AND MAINTENANCE, MARCH 1, 1980, 10 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-4	02/17/80	S	.0	H	5				POWER WAS REDUCED TO ALLOW ADJUSTMENT OF THE CONTROL ROD PATTERN.

SUMMARY

OPERATION WAS ROUTINE DURING THE REPORT PERIOD, WITH A BRIEF LOAD REDUCTION ON 02/17/80 FOR CONTROL ROD ADJUSTMENT.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

CRYSTAL RIVER 3

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1. Location (state/county) (nearest population center): Florida/Citrus, 7 miles NW of Crystal River
2. Electric Reliability Council: Southeastern
3. Type of Reactor: PWR
4. Power Level (MWT): 2452
5. Design Electrical Rating (MWE Net): 825
6. Date of Initial Criticality: January 14, 1977
7. Date of Commercial Operation: March 13, 1977
8. Condenser Cooling Method: Once Through
9. Condenser Cooling Water Source: Gulf of Mexico

Utility & Contractor Information

10. Licensee: Florida Power Corporation
11. Corporate Address: 3201 34th Street, South, St. Petersburg, Florida 33733
12. Corporate Contact: J. Hancock
13. Architect/Engineer: Gilbert Associates, Inc.
14. Nuclear Steam Supply System: B&W
15. Constructor: J. A. Jones Construction Co.

Regulatory Information

16. IE Region responsible: II
17. IE Branch Chief: R. C. Lewis
18. IE Senior Inspector: R. Martin
19. IE Principal Inspector: L. Smith
20. Licensing Project Manager: M. B. Fairtile
21. Docket Number: 50-302
22. License Number/Date of Issue: DPR-72, 12/3/76
23. Public Document Room: Crystal River Public Library, 668 N.W. First, Crystal River, Florida 32639

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO TECHNICAL SPECIFICATION 4.3.3.2B THE INCORE DETECTOR CALIBRATION WAS NOT CONDUCTED WITHIN THE ALLOWABLE TIME FRAME. (8003 INFRACTION)

AS REQUIRED BY 10CFR50, APPENDIX B, CRITERION V, "ACTIVITIES AFFECTING QUALITY SHALL BE PRESCRIBED BY...PROCEDURES...AND SHALL BE ACCOMPLISHED IN ACCORDANCE WITH THESE ..PROCEDURES...". THE LICENSEE'S QOP-9.0 REQUIRES PIPES TO HAVE ENDS COVERED AND STAINLESS NOT IN CONTACT WITH CARBON STEEL. ON 1-29-80 IT WAS NOTED THAT PIPE ENDS WERE NOT COVERED AND STAINLESS STEEL PIPE WAS STORED ON A CARBON RACK. (8007 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

+ LOSS OF POWER TO NNI UNDER INVESTIGATION.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

+ UNIT SHUT DOWN PENDING REVIEW OF TRANSIENT.

PLANT STATUS:

+ RECOVERING FROM FEBRUARY 26 EVENT CAUSED BY LOSS OF POWER TO NNI.

+ LAST IE SITE INSPECTION DATE: FEBRUARY 25 - MARCH 4 1980

+ INSPECTION REPORT NO: 50-302/80-11

SUMMARY

INSPECTION SUMMARY

+ INSPECTION JANUARY 16-18 AND 24-25 (80-02): THIS SPECIAL, ANNOUNCED INSPECTION INVOLVED 24 INSPECTOR-HOURS ON SITE AND GILBERT ASSOCIATES, INC., OFFICES IN THE AREAS OF SEISMIC ANALYSIS FOR AS-BUILT SAFETY-RELATED PIPING SYSTEMS (IE BULLETIN 79-14) AND OF CONCRETE EXPANSION ANCHOR INSTALLATION (IE BULLETIN 79-02). OF THE TWO AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION FEBRUARY 5-8 (80-05): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 21 INSPECTOR-HOURS ON SITE IN THE AREAS OF PLANT OPERATIONS AND REVIEW OF PREVIOUSLY IDENTIFIED INSPECTOR ITEMS. OF THE TWO AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION JANUARY 14-17 (80-03): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 14 INSPECTOR-HOURS ON SITE IN THE AREAS OF PLANT OPERATIONS, REVIEW OF PREVIOUSLY IDENTIFIED INSPECTOR ITEMS, DISCUSSION OF PART 21 REPORTS, AND REVIEW OF SURVEILLANCE PROCEDURES. OF THE FOUR AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN THREE AREAS. ONE APPARENT ITEM OF NONCOMPLIANCE WAS IDENTIFIED IN ONE AREA (INFRACTION - FAILURE TO CONDUCT SURVEILLANCE AS REQUIRED BY TECHNICAL SPECIFICATIONS).

INSPECTION JANUARY 21-25 (80-04): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 30 INSPECTOR-HOURS ON SITE IN THE AREA OF PLANT OPERATIONS. OF THE ONE AREA INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION JANUARY 28 - FEBRUARY 1 (80-07): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 30 INSPECTOR-HOURS ON SITE IN THE AREAS OF OPERATIONS, PLANT TOUR, AND FOLLOWUP OF PREVIOUSLY IDENTIFIED ITEMS. OF THE THREE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN TWO AREAS; ONE ITEM OF NONCOMPLIANCE WAS FOUND IN ONE AREA (INFRACTION - FAILURE TO FOLLOW PROCEDURES).

REPORTS RECEIVED FROM LICENSEE

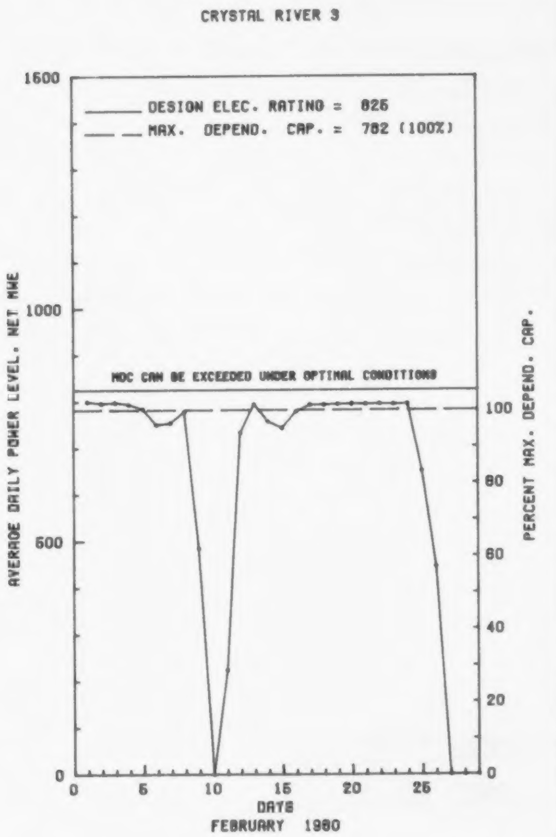
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-109/ 03L-0	12/21/79	01/17/80	30 DAY LER	IODINE 131 SPIKING IN REACTOR COOLANT SYSTEM FOLLOWING REACTOR TRIP.
79-110/ 03L-0	12/23/79	01/17/80	30 DAY LER	INOPERABLE DECAY HEAT COOLER OUTLET TEMPERATURE INDICATOR.
79-111/ 03L-0	12/24/79	01/21/80	30 DAY LER	MAKEUP PUMP 1B DISCHARGE DRAIN LINE WELD LEAK.
79-112/ 04X-0	01/07/80	01/21/80	ENVIRONMENTAL	ENVIRONMENTAL SAMPLE LLD'S EXCESSIVE.
79-113/ 03L-0	12/27/79	01/25/80	30 DAY LER	RADIATION MONITOR RMA-5 FOR CONTROL COMPLEX FAILED.
79-114/ 04L-0	01/24/80	02/06/80	ENVIRONMENTAL	EXCESSIVE DEVIATION IN THE METABOLIC FUNCTIONS STUDY.
80-001/ 01T-0	01/07/80	01/21/80	14 DAY LER	UNIT ENTERED INTO OPERATIONAL MODE WHILE IN AN ACTION STATEMENT OF A LCO-CONTAINMENT ISOLATION VALVE CAV-1 INOPERABLE.
80-002/ 03L-0	01/05/80	02/04/80	30 DAY LER	IODINE SPIKING IN THE RCS FOLLOWING REACTOR SHUTDOWN.
80-003/ 03L-0	01/04/80	02/04/80	30 DAY LER	PRESSURIZER STEAM SPACE SAMPLE VALVE CAV-1 FAILED TO CLOSE.

REVIEWED BY OMPA LOEB: P.E. Bobe **DATE:** 4/10/80

REVIEWED BY NRR DOR: Morton B. Fairtile **DATE:** 4/11/80

CRYSTAL RIVER 3

AVERAGE POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-302
2. Utility Contact: M.W. CULVER (904) 795-6486
3. Licensed Therma. Power (MWt): 2452
4. Nameplate Rating (Gross MWe): 989 x 0.9 = 890
5. Design Electrical Rating (Net MWe): 825
6. Maximum Dependable Capacity (Gross MWe): 821
7. Maximum Dependable Capacity (Net MWe): 782
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
ITEMS 6&7 REEVALUATED TO REFLECT ACTUAL PERFORMANCE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	696.0	1,440.0	26,016.0
12. Hours Reactor Was Critical	572.5	1,305.5	16,455.7
13. Reactor Reserve Shutdown Hours	.0	.0	1,116.9
14. Hours Generator On-Line	568.0	1,297.4	15,997.7
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,345,568	3,085,927	35,402,838
17. Gross Electrical Energy (MWH)	458,458	1,058,396	12,030,164
18. Net Electrical Energy (MWH)	437,893	1,009,335	11,400,975
19. Unit Service Factor	81.6	90.1	61.5
20. Unit Availability Factor	81.6	90.1	61.5
21. Unit Capacity Factor (MDC Net)	80.5	89.6	56.0
22. Unit Capacity Factor (DER Net)	76.3	85.0	53.1
23. Unit Forced Outage Rate	18.4	9.0	31.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: [illegible]

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-6	02/09/80	F	46.5	A	1		RB	CRDRVE	ASYMETRIC FAULT ALARM RECEIVED; POWER REDUCED TO 60% FOR 8.5 HRS., THEN SHUTDOWN FOR REPAIR WHEN ROD 6-1 DROPPED TO ITS INLIMIT THE 2ND TIME. DROPPED ROD DUE TO STATOR FAILURE.
80-7	02/25/80	F	.0	A	5		CH	PIPEXX	REDUCED POWER TO 50% TO REPAIR A CRACK IN A WELD ON THE "B" MFWP RECIRC. LINE.
80-8	02/26/80	F	81.5	A	3	80-010	IF	INSTRU	LOSS OF POWER TO PART OF THE PLANT INSTRUMENTATION SYSTEM RESULTING IN A FORCED SHUTDOWN.

SUMMARY

THE UNIT WAS TAKEN OFF-LINE ON 02/09/80 DUE TO STATOR FAILURE, AND ON 02/26/80 BECAUSE OF LOSS OF POWER TO PART OF THE PLANT INSTRUMENTATION SYSTEM. LOAD WAS REDUCED ON 02/25/80 TO REPAIR A CRACK IN THE "B" MAIN FEEDWATER PUMP RECIRCULATION LINE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

DAVIS-BESSE 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Ohio/Ottawa 21 miles E of Toledo
2.	Electric Reliability Council	ECAR
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2772
5.	Design Electrical Rating (MWE Net)	906
6.	Date of Initial Criticality	August 12, 1977
7.	Date of Commercial Operation	11/21/77 (25%); 12/19/77 (40%) 01/23/78 (75%); 07/31/78 (100%)
8.	Condenser Cooling Method	Cooling Tower
9.	Condenser Cooling Water Source	Lake Erie

Utility & Contractor Information

10.	Licensee	Toledo Edison Co.
11.	Corporate Address	Edison Plaza, 300 Madison Avenue Toledo, Ohio 43652
12.	Corporate Contact	R. P. Crouse, V. P., Nuclear (419)259-5221
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	B&W
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Senior Inspector	R. F, Warnick
19.	IE Resident Inspector	L. Reyes (419) 898-2776
20.	Licensing Project Manager	D. Garner
21.	Docket Number	50-346
22.	License Number/Date of Issue	NPF-3, 4/22/77
23.	Public Document Room	Ida Rupp Public Library 310 Madison Street Port Clinton, OH 43452

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

- Routine operation.

- LAST IE SITE INSPECTION DATE: November 21; December 12-14, 1979 and January 25, 1980

- INSPECTION REPORT NO: 79-33

SUMMARY

INSPECTION SUMMARY

- Inspection On December 3-7, 11-13 and 19-20, (79-30): Announced special inspection of the licensee's implementation of the Quality Assurance Program in the following areas: QA/QC administration; qualification of personnel; design changes and modifications; records control; receipt, storage and handling; tests and experiments; procurement control; document control; offsite review committee; audits; test and measurement equipment; survaillance and calibration; maintenance; housekeeping and cleanliness; and offsite support staff. The inspection involved 222 inspector hours onsite by three NRC inspectors. Of the 15 areas inspected, no apparent items of noncompliance or deviations were identified in 12 areas, three apparent items of noncompliance were identified in three areas (failure to comply with record control requirements; failure to maintain control of drawings; and failure to conduct required reviews).

- Inspection on December 3-7, 10-14, 17-21, 26-28 and 31, (79-31): A routine, unannounced inspection of plant operations, physical protection (security organization, physical barriers, access control, communications), followup of licensee event reports, the modification of the hot leg RTD's, nuclear instrumentation calibration procedures and HPI line restraint work. The inspection involved 175 inspector hours onsite by the Resident Inspector. No items of noncompliance or deviations were identified.

- Inspection on November 21; December 12-14, 1979 and January 25, 1980, (79-33): Inspection of RTD connection repair at Hot Leg risers; licensee implementation of IEB 79-02 and IEB 79-14; and snubber installations. The inspection involved 26 inspector hours onsite, at the licensee corporate office, and at the NRC office by one NRC inspector. No items of noncompliance or deviations were identified.

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
NP 80-01	01/16/80	02/12/80	30 Day LER	Discharge temperature for station liquid effluents exceeded limit.
80-02/ 03L-0	01/03/80	01/30/80	30 Day LER	Demin. water valve did not indicate closed and SAM light was not indicating properly.
80-03/ 03L-0	01/03/80	01/30/80	30 Day LER	Auxiliary feedwater pump encountered problems with turbine governor.
80-04/ 03L-0	01/07/80	02/04/80	30 Day LER	Control rod absolute position indication declared inoperable.
80-05/ 03L-0	01/10/80	02/06/80	30 Day LER	Tech Spec late date exceeded on boron injection flowpath test.
80-06/ 03L-0	01/12/80	02/07/80	30 Day LER	Fire damper inoperable.
80-07/ 03L-0	01/16/80	02/12/80	30 Day LER	Increased surveillance frequency requirement overlooked by operator.
80-08/ 03L-0	01/16/80	02/13/80	30 Day LER	A gap found around 1 1/2 inch conduit which penetrated a fire wall.
80-09/ 03L-0	01/22/80	02/15/80	30 Day LER	NI-7 indication on computer group reading less than heat balance power.

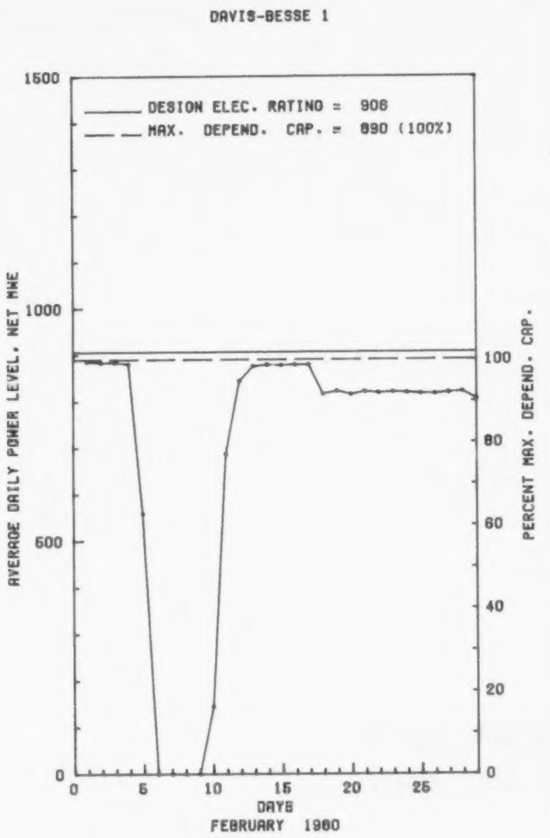
==

REVIEWED BY OMPA LOEB: **DATE:** 4/10/80

REVIEWED BY NRR DOR: **DATE:** 4/11/80

DAVIS-BESSE 1

AVERAGE POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-346
2. Utility Contact: E. Caba (419) 259-5000 x236
3. Licensed Thermal Power (MWt): 2772
4. Nameplate Rating (Gross MWe): 1069 x 0.9 = 962
5. Design Electrical Rating (Net MWe): 905
6. Maximum Dependable Capacity (Gross MWe): 934
7. Maximum Dependable Capacity (Net MWe): 890
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: ITEMS 6&7, FIRST CAPACITY TEST, WERE USING DER
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	696.0	1,440.0	21,965.0
12. Hours Reactor Was Critical	614.6	1,238.7	12,202.9
13. Reactor Reserve Shutdown Hours	.0	.0	2,875.8
14. Hours Generator On-Line	587.4	1,180.5	11,055.2
15. Unit Reserve Shutdown Hours	.0	.0	1,728.2
16. Gross Thermal Energy (MWH)	1,495,351	3,065,778	23,265,285
17. Gross Electrical Energy (MWH)	505,400	1,032,449	7,755,960
18. Net Electrical Energy (MWH)	476,480	973,117	7,143,695
19. Unit Service Factor	84.4	82.0	51.8
20. Unit Availability Factor	84.4	82.0	60.5
21. Unit Capacity Factor (MDC Net)	76.9	75.9	39.7
22. Unit Capacity Factor (DER Net)	75.6	74.6	39.0
23. Unit Forced Outage Rate	15.6	18.0	26.7

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUELING OUTAGE - APRIL, 1980 - 12 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
1	02/05/80	F	108.6	A	3				REACTOR TRIP ON HIGH PRESSURE CAUSED APPARENTLY BY A FALSE INDICATION OF TURBINE OVERSPEED.

SUMMARY

OPERATION WAS ROUTINE DURING THE REPORT PERIOD, EXCEPT FOR A REACTOR TRIP ON HIGH REACTOR COOLANT SYSTEM PRESSURE. SINCE THIS WAS APPARENTLY IN RESPONSE TO A FALSE INDICATION OF TURBINE OVERSPEED, STATION PERSONNEL REPLACED ALL CIRCUIT BOARDS WHICH COULD HAVE BEEN THE SOURCE OF THE FAULT SIGNAL. DURING THIS OUTAGE, CONDENSER TUBE LEAKS WERE ALSO LOCATED AND PLUGGED.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examinatior
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

DRESDEN 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Illinois/Grundy 9 miles E of Morris
2.	Electric Reliability Council	Main
3.	Type of Reactor	BWR
4.	Power Level (MWT)	700
5.	Design Electrical Rating (MWE Net)	200
6.	Date of Initial Criticality	October 15, 1959
7.	Date of Commercial Operation	July 4, 1960
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Kankakee River

Utility & Contractor Information

10.	Licensee	Commonwealth Edison Co.
11.	Corporate Address	P. O. Box 767 Chicago, Illinois 60690
12.	Corporate Contact	C. Reed
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	GE
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Senior Inspector	R. Spessard
19.	IE Resident Inspector	J. L. Barker (815)-942-9267
20.	Licensing Project Manager	S. J. Nowicki
21.	Docket Number	50-010
22.	License Number/Date of Issue	DPR-2 issued 9/28/59
23.	Public Document Room	Morris Public Library 604 Liberty Street Morris, Illinois 60451

INSPECTION STATUS

ENFORCEMENT STATUS

UNIT 1 TECHNICAL SPECIFICATIONS, SECTION 6.2.B REQUIRES THAT RADIATION CONTROL PROCEDURES BE MAINTAINED, MADE AVAILABLE TO ALL STATION PERSONNEL, AND ADHERED TO. RADIATION CONTROL STANDARDS PROCEDURE 37-1-E-3, "WORK IN CONTROLLED AREAS (RADIATION AREAS AND HIGH RADIATION AREAS)," REQUIRES THAT PERSONNEL NOT EAT, DRINK, SMOKE, OR CHEW IN THOSE CONTROLLED AREAS. CONTRARY TO THE ABOVE, ON DECEMBER 20, 1979, WHILE MAKING A ROUTINE TOUR OF THE UNIT 1 TURBINE BUILDING (A POSTED RADIATION AREA), THE NRC INSPECTOR OBSERVED EVIDENCE OF EATING, DRINKING, SMOKING, AND CHEWING (I.E., THE PRESENCE OF NUMEROUS CIGARETTE BUTTS, EMPTY SOFT DRINK CANS, EMPTY CANDY WRAPPERS, AND EMPTY CHEWING GUM WRAPPERS) IN THIS RADIATION AREA. THIS IS A REPETITIVE ITEM OF NONCOMPLIANCE SINCE THE SAME PROBLEM WAS IDENTIFIED PREVIOUSLY IN NRC INSPECTION REPORT NO. 50-010/79-19, DATED OCTOBER 18, 1979. (7925 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

FACILITY ITEMS

+ Radwaste shipments to Barnwell, South Carolina were resumed on November 28, 1979 following rescission of a shipping ban imposed by the State of South Carolina. (79-23)

MANAGERIAL ITEMS

+ The licensee has committed to review/revise the procedures and training program for processing, packaging, transfer, and transport of radioactive material by early 1980. (79-23)

PLANT STATUS

Unit 1 shutdown on October 31, 1978 for an extended outage to upgrade ECCS, chemical cleaning and refueling. Outage expected to last a minimum of 18 months.

+LAST IE SITE INSPECTION DATE: January 10, 1980

+ INSPECTION REPORT NO: 80-02

SUMMARY

INSPECTION SUMMARY

+ Inspection on October 9 and November 28, (79-23): Nonroutine, announced inspection of licensee actions taken in response to IE Bulletin 79-19, "Packaging of Low Level Radioactive Waste for Transport and Burial" and, as agreed with the State of South Carolina, verification that identified packaging discrepancies were corrected before resumption of radwaste shipments to Barnwell, South Carolina. The inspection involved 14 inspector hours onsite by one NRC inspector. Of the 10 areas inspected, no items of noncompliance or deviations were found in nine areas. One apparent item of noncompliance was found in the remaining area (failure to follow procedures).

+ Inspection on December 3, 1979 through January 4, 1980, (79-25): Routine, unannounced resident inspection of maintenance activities; plant operations; physical protection-security organization, physical barriers, access control (identification, authorization, badging, search, and escorting), and communications; licensee event reports; surveillance of safety related systems/components required by technical specifications; calibration of safety related components required by technical specifications; radioactive waste system-operations; surveillance of core power distribution limits; thermal power evaluation; radiation protection-operations; calibration of LPRM system; APRM calibration; calibration of safety related components not required by technical specifications; critical fire protection area inspection; and, design, design changes, and modifications. The inspection involved 122 inspector hours onsite by two NRC inspectors. Of the 18 areas inspected, there were no items of noncompliance identified in 17 areas. There was one item of noncompliance (failure to follow procedures) identified in one area.

+ Inspection on January 8-9, (80-01): Routine, announced inspection of Units 2 and 3 Emergency Diesel Generator modifications; modification of electrical equipment for Unit 1 relative to compliance to IEEE-279; and, followup of previously identified unresolved items. The inspection involved 14 inspector hours onsite by one NRC inspector. No items of noncompliance were identified.

+ Inspection on January 10, (80-02): A special announced meeting was conducted to discuss recent personnel errors at Dresden Nuclear Power Station; and, Cooling Lake leakage at Dresden Nuclear Power Station. The meeting involved 12 inspector hours at the corporate office by six NRC personnel.

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
NONE				

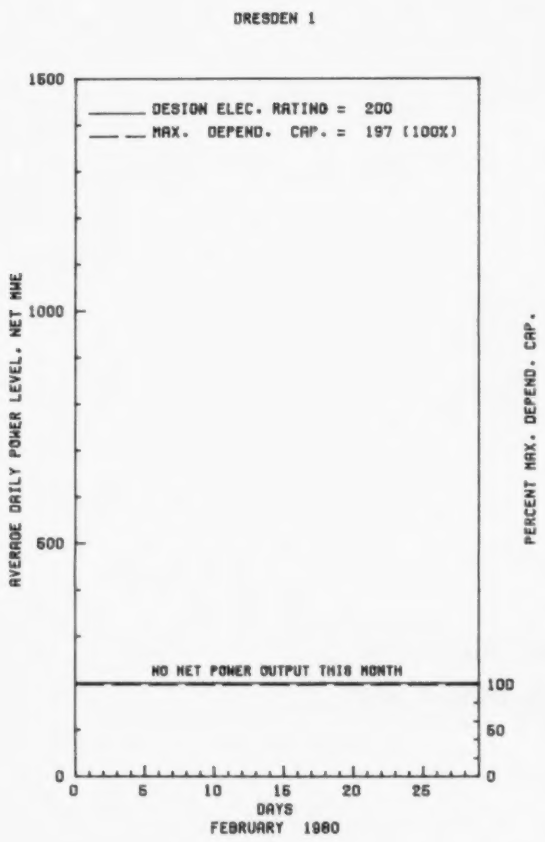
REVIEWED BY OMPA LOEB: B Muranaka **DATE:** 4/10/80

REVIEWED BY NRR DOR: R. P. Silver for S. Nowicki **DATE:** 4/14/80

GPO 873-667

DRESDEN 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-010
2. Utility Contact: S. A. Cieszkiewicz (815) 942-2920
3. Licensed Thermal Power (MWt): 700
4. Nameplate Rating (Gross MWe): 245.5 x 0.85 = 209
5. Design Electrical Rating (Net MWe): 200
6. Maximum Dependable Capacity (Gross MWe): 205
7. Maximum Dependable Capacity (Net MWe): 197
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours In Reporting Period	696.0	1,440.0	174,240.0
12. Hours Reactor Was Critical	.0	.6	113,153.9
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	.0	.0	111,196.2
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	0	56,016,747
17. Gross Electrical Energy (MWH)	0	0	16,799,254
18. Net Electrical Energy (MWH)	-758	-1,633	15,805,134
19. Unit Service Factor	.0	.0	63.3
20. Unit Availability Factor	.0	.0	63.8
21. Unit Capacity Factor (MDC Net)	.0	.0	56.1
22. Unit Capacity Factor (DER Net)	.0	.0	55.4
23. Unit Forced Outage Rate	.0	.0	11.2

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 08/01/81

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
8	10/31/78	S	696.0	C	4			FUELXX	REFUEL AND CHEMICAL CLEANING OUTAGE.

SUMMARY

THE UNIT SHUTDOWN THROUGHOUT THE REPORT PERIOD FOR REFUELING AND CHEMICAL CLEANING.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

DRESDEN 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Illinois/Grundy 9 miles E of Morris
2.	Electric Reliability Council	Main
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2527
5.	Design Electrical Rating (MWE Net)	794
6.	Date of Initial Criticality	January 7, 1970
7.	Date of Commercial Operation	JULY, 1970
8.	Condenser Cooling Method	Cooling Lake
9.	Condenser Cooling Water Source	Kankakee and Des Plaines Rivers

Utility & Contractor Information

10.	Licensee	Commonwealth Edison Co.
11.	Corporate Address	P. O. Box 767 Chicago, Illinois 60690
12.	Corporate Contact	C. Reed
13.	Architect/Engineer	Sargent & Lundy
14.	Nuclear Steam Supply System	GE
15.	Constructor	United Engineers & Constructors, Inc.

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Senior Inspector	R. Spessard
19.	IE Resident Inspector	J. L. Barker (815)-942-5267
20.	Licensing Project Manager	S. J. Nowicki
21.	Docket Number	50-237
22.	License Number/Date of Issue	DPR-19 issued 12/22/69
23.	Public Document Room	Morris Public Library 604 Liberty Street Morris, Illinois 60451

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENTS

- A new cement radwaste solidification systems has replaced an outdated solidification system. (79-27)
- Modifications to the emergency diesel generators have been completed. The modifications involved: (1) alarm for manual air shutoff valve placed in the closed positions; (2) alarm for bus tie breaker between the diesel generator and its emergency bus placed in the test position; (3) circuit revisions to allow multiple diesel air start attempts; and (4) alarm for incorrect droop setting. (80-01)

FACILITY ITEMS

- Radwaste shipments to Barnwell, South Carolina were resumed on November 28, 1979 following rescission of a shipping ban imposed by the State of South Carolina. (79-27)

MANAGERIAL ITEMS

- The licensee has committed to review/revise the procedures and training program for processing, packaging, transfer, and transport of radioactive material by early 1980. (79-27)

PLANT STATUS

Unit operating routinely.

*LAST IE SITE INSPECTION DATE: January 10, 1980

- INSPECTION REPORT NO: 80-02

SUMMARY

INSPECTION SUMMARY

- Inspection on October 9 and November 28, (79-27): Nonroutine, announced inspection of licensee actions taken in response to IE Bulletin 79-19, "Packaging of Low Level Radioactive Waste for Transport and Burial" and, as agreed with the State of South Carolina, verification that identified packaging discrepancies were corrected before resumption of radwaste shipments to Barnwell, South Carolina. The inspection involved 14 inspector hours onsite by one NRC inspector. Of the 10 areas inspected, no items of noncompliance or deviations were found in nine areas. One apparent item of noncompliance was found in the remaining area (failure to follow procedures).
- Inspection on December 3, 1979 through January 4, 1980, (79-29): Routine, unannounced resident inspection of maintenance activities; plant operations; physical protection-security organization, physical barriers, access control (identification, authorization, badging, search, and escorting), and communications; licensee event reports; surveillance of safety related systems/components required by technical specifications; calibration of safety related components required by technical specifications; radioactive waste system-operations; surveillance of core power distribution limits; thermal power evaluation; radiation protection-operations; calibration of LPRM system; APRM calibration; calibration of safety related components not required by technical specifications; critical fire protection area inspection; and, design, design changes, and modifications. The inspection involved 122 inspector hours onsite by two NRC inspectors. Of the 18 areas inspected, there were no items of noncompliance identified.
- Inspection on January 8-9, (80-01): Routine, announced inspection of Units 2 and 3 Emergency Diesel Generator modifications; modification of electrical equipment for Unit 1 relative to compliance to IEEE-279; and, followup of previously identified unresolved items. The inspection involved 14 inspector hours onsite by one NRC inspector. No items of noncompliance were identified.
- Inspection on January 10, (80-02): A special announced meeting was conducted to discuss recent personnel errors at Dresden Nuclear Power Station; and, Cooling Lake leakage at Dresden Nuclear Power Station. The meeting involved 12 inspector hours at the corporate office by six NRC personnel.

REPORTS RECEIVED FROM LICENSEE

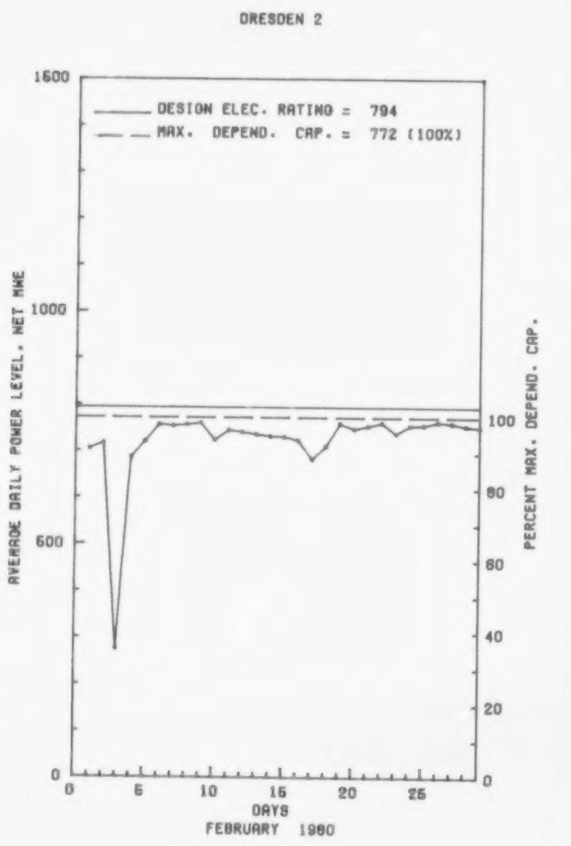
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
80-01/ 03L-0	01/06/80	01/30/80	30 Day LER	Number 3 turbine control valve inoperable.
80-02/ 03L-0	01/04/80	01/30/80	30 Day LER	Surveillance of main steam line low pressure isolation not completed within TS periodicity.
80-03/ 03L-0	01/14/80	02/13/80	30 Day LER	Reactor water level switch (LIS-2-263-58A) tripped in excess of TS limits.
80-04/ 03L-0	01/18/80	01/30/80	30 Day LER	MSIV 2-203-1B closed in less than time specified.
80-06/ 03L-0	01/19/80	02/15/80	30 Day LER	Curie content of "A" Waste Sample Tank greater than TS limits.
80-07/ 03L-0	01/25/80	02/19/80	30 Day LER	Erratic oxygen reading received in oxygen analyzer for drywell atmosphere.

REVIEWED BY OMPA LOEB: DATE: 4/10/80

REVIEWED BY NRR DOR: R. O. Silver for S. Nowicki DATE: 4/14/80

DRESDEN 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-237
2. Utility Contact: S. A. Cieszkiewicz (815) 942-2920 x263
3. Licensed Thermal Power (MWt): 2527
4. Nameplate Rating (Gross MWe): 920 x 0.9 = 828
5. Design Electrical Rating (Net MWe): 794
6. Maximum Dependable Capacity (Gross MWe): 812
7. Maximum Dependable Capacity (Net MWe): 772
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	696.0	1,440.0	85,896.0
12. Hours Reactor Was Critical	690.0	1,424.7	66,161.4
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	685.3	1,415.0	62,677.7
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,634,267	3,304,254	124,946,086
17. Gross Electrical Energy (MWH)	526,209	1,063,530	40,070,151
18. Net Electrical Energy (MWH)	500,629	1,011,949	37,900,061
19. Unit Service Factor	98.5	98.3	73.0
20. Unit Availability Factor	98.5	98.3	73.0
21. Unit Capacity Factor (MDC Net)	93.2	91.0	57.2
22. Unit Capacity Factor (DER Net)	90.6	88.5	55.6
23. Unit Forced Outage Rate	1.5	1.7	13.2

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
2	02/03/80	F	10.7	A	3			HTEXCH	RX SCRAMMED ON LOW VACUUM WHILE TRANSFERRING MAX RECYCLE REBOILER RELIEF DISCHARGE FROM UNIT 3 CONDENSER TO UNIT 2 CONDENSER.

SUMMARY

OPERATION WAS ROUTINE DURING THE REPORT PERIOD, WITH A REACTOR SCRAM ON 02/03/80 DUE TO LOW VACUUM WHILE TRANSFERRING THE MAX-RECYCLE REBOILER RELIEF DISCHARGE LINE FROM THE UNIT 3 CONDENSER TO THE UNIT 2 CONDENSER.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

DRESDEN 3

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Illinois/Grundy 9 miles E of Morris
2.	Electric Reliability Council	Main
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2527
5.	Design Electrical Rating (MWE Net)	794
6.	Date of Initial Criticality	January 31, 1971
7.	Date of Commercial Operation	November 16, 1971
8.	Condenser Cooling Method	Cooling Lake
9.	Condenser Cooling Water Source	Kankakee and Des Plaines Rivers

Utility & Contractor Information

10.	Licensee	Commonwealth Edison Co.
11.	Corporate Address	P. O. Box 767 Chicago, Illinois 60690
12.	Corporate Contact	D. L. Peoples
13.	Architect/Engineer	Sergent & Lundy
14.	Nuclear Steam Supply System	GE
15.	Constructor	United Engineers & Constructors, Inc.

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Senior Inspector	R. Spessard
19.	IE Resident Inspector	J. L. Barker 815-942-9267
20.	Licensing Project Manager	R. Bevan
21.	Docket Number	50-249
22.	License Number/Date of Issue	DPR-25 issued 1/12/71
23.	Public Document Room	Morris Public Library 604 Liberty Street Morris, Illinois 60451

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENTS

- A new cement radwaste solidification systems has replaced an outdated solidification system. (79-25)
- Modifications to the emergency diesel generators have been completed. The modifications involved: (1) alarm for manual air shutoff valve placed in the closed positions; (2) alarm for bus tie breaker between the diesel generator and its emergency bus placed in the test position; (3) circuit revisions to allow multiple diesel air start attempts; and (4) alarm for incorrect droop setting. (80-01)

FACILITY ITEMS

- Radwaste shipments to Barnwell, South Carolina were resumed on November 28, 1979 following rescission of a shipping ban imposed by the State of South Carolina. (79-25)

MANAGERIAL ITEMS

- The licensee has committed to review/revise the procedures and training program for processing, packaging, transfer, and transport of radioactive material by early 1980. (79-25)

PLANT STATUS

- The unit was shut down on February 2, 1980 due to a failed jet pump. The licensee commenced a 12-week refueling/maintenance outage 6 weeks early as a results of this event.

+LAST IE SITE INSPECTION DATE: January 10, 1980

- INSPECTION REPORT NO: 80-02

SUMMARY

INSPECTION SUMMARY

- Inspection on October 9 and November 28, (79-25): Nonroutine, announced inspection of licensee actions taken in response to IE Bulletin 79-19, "Packaging of Low Level Radioactive Waste for Transport and Burial" and, as agreed with the State of South Carolina, verification that identified packaging discrepancies were corrected before resumption of radwaste shipments to Barnwell, South Carolina. The inspection involved 14 inspector hours onsite by one NRC inspector. Of the 10 areas inspected, no items of noncompliance or deviations were found in nine areas. One apparent item of noncompliance was found in the remaining area (failure to follow procedures).
- Inspection on December 3, 1979 through January 4, 1980, (79-27): Routine, unannounced resident inspection of maintenance activities; plant operations; physical protection-security organization, physical barriers, access control (identification, authorization, badging, search, and escorting), and communications; licensee event reports; surveillance of safety related systems/components required by technical specifications; calibration of safety related components required by technical specifications; radioactive waste system-operations; surveillance of core power distribution limits; thermal power evaluation; radiation protection-operations; calibration of LPRM system; APRM calibration; calibration of safety related components not required by technical specifications; critical fire protection area inspection; and, design, design changes, and modifications. The inspection involved 122 inspector hours onsite by two NRC inspectors. Of the 18 areas inspected, there were no items of noncompliance identified.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
80-01/ 03L-0	01/16/80	02/11/80	30 Day LER	Drywell oxygen analyzer inoperable.
80-02/ 03L-0	01/22/80	02/14/80	30 Day LER	LPCI valve, MO-3-1501-11B inoperable.
80-03/ 03L-0	01/28/80	02/11/80	30 Day LER	3B core spray relief valve found leaking.

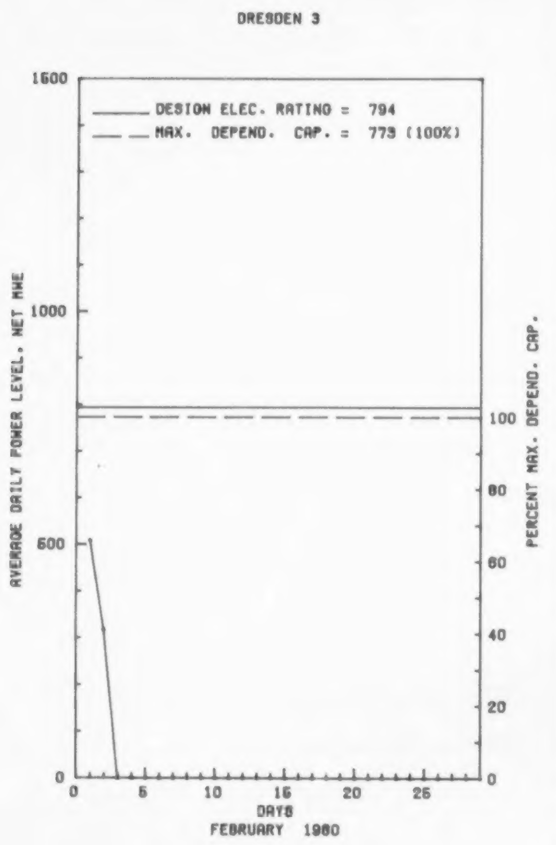
- Inspection on January 8-9, (80-01): Routine, announced inspection of Units 2 and 3 Emergency Diesel Generator modifications; modification of electrical equipment for Unit 1 relative to compliance to IEEE-279; and, followup of previously identified unresolved items. The inspection involved 14 inspector hours onsite by one NRC inspector. No items of noncompliance were identified.
- Inspection on January 10, (80-02): A special announced meeting was conducted to discuss recent personnel errors at Dresden Nuclear Power Station; and, Cooling Lake leakage at Dresden Nuclear Power Station. The meeting involved 12 inspector hours at the corporate office by six NRC personnel.

REVIEWED BY OMPA LOEB: __________ **DATE:** 4/10/80

REVIEWED BY NRR DOR: __________ **DATE:** 4/11/80

DRESDEN 3

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-249
2. Utility Contact: S. A. Cieszkiewicz (815) 942-2920 x263
3. Licensed Thermal Power (MWt): 2527
4. Nameplate Rating (Gross MWe): 920 x 0.9 = 828
5. Design Electrical Rating (Net MWe): 794
6. Maximum Dependable Capacity (Gross MWe): 812
7. Maximum Dependable Capacity (Net MWe): 773
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	696.0	1,440.0	75,481.0
12. Hours Reactor Was Critical	41.0	785.0	56,550.4
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	34.7	775.7	54,103.3
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	52,593	1,327,726	103,599,394
17. Gross Electrical Energy (MWH)	16,693	424,659	33,854,333
18. Net Electrical Energy (MWH)	11,749	396,289	32,089,580
19. Unit Service Factor	5.0	54.1	71.7
20. Unit Availability Factor	5.0	54.1	71.7
21. Unit Capacity Factor (MDC Net)	2.2	35.6	55.0
22. Unit Capacity Factor (DER Net)	2.1	34.7	53.5
23. Unit Forced Outage Rate	38.0	2.7	16.3

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 05/21/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
1	02/02/80	F	21.3	A	1	LER 80-4/03L		PUMPXX	JET PUMP #13 WAS FOUND TO BE INOPERABLE.
2	02/03/80	5	640.0	C	1		RC	FUELXX	UNIT COMMENCED ITS SCHEDULED REFUELING OUTAGE.

SUMMARY

THE UNIT BEGAN ITS SCHEDULED REFUELING OUTAGE ON 02/03/80, AFTER IT WAS TAKEN OFF-LINE ON 02/02/80 DUE TO AN INOPERABLE JET PUMP.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

NRC POWER RESTRICTIONS: NONE

DUANE ARNOLD

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Iowa/Linn 8 miles NW of Cedar Rapids
2.	Electric Reliability Council	N/A
3.	Type of Reactor	BWR
4.	Power Level (MWT)	1658
5.	Design Electrical Rating (MWE Net)	538
6.	Date of Initial Criticality	March 23, 1974
7.	Date of Commercial Operation	February 1, 1975
8.	Condenser Cooling Method	Cooling Towers
9.	Condenser Cooling Water Source	Cedar Rapids River Circulating Water System

Utility & Contractor Information

10.	Licensee	Iowa Electric Light and Power
11.	Corporate Address	IE Towers, P. O. Box 351 Cedar Rapids, Iowa 52406
12.	Corporate Contact	D. Arnold, President
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	GE
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Senior Inspector	R. Spessard
19.	IE Principal Inspector	G. Wright
20.	Licensing Project Manager	T. Kevern
21.	Docket Number	50-331
22.	License Number/Date of Issue	DPR-49 issued 2/22/74
23.	Public Document Room	Reference Service Cedar Rapids Public Library 428 Third Avenue, S.E. Cedar Rapids, Iowa 52401

INSPECTION STATUS

ENFORCEMENT STATUS

ENVIRONMENTAL TECHNICAL SPECIFICATION 2.2.1 REQUIRES THAT RESIDUAL CHLORINE CONCENTRATIONS IN PLANT COOLING TOWER EFFLUENT BE LIMITED TO 0.1 MG/L OR LESS AT ALL TIMES. CONTRARY TO THE ABOVE, ON NOVEMBER 8, 1979, WATER DISCHARGED FROM PLANT COOLING TOWERS CONTAINED RESIDUAL CHLORINE CONCENTRATIONS IN EXCESS OF 0.1 MG/L.
(7931 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

Shutdown for refueling.

+LAST IE SITE INSPECTION DATE: October 9-12, 18-19, 25-26; November 1-2 and 20, 1979

+ INSPECTION REPORT NO: 79-25

SUMMARY

INSPECTION SUMMARY

+ Inspection on October 9-12, 18-19, 25-26; November 1-2 and 20, (79-25): Announced special inspection of the licensee's implementation of the Quality Assurance Program in the following areas: QA/QC administration; qualification of personnel; design changes and modifications; records control; receipt, storage and handling; tests and experiments; procurement control; document control; offsite review committee; audits; test and measurement equipment; surveillance and calibration; maintenance; housekeeping and cleanliness; and offsite support staff. The inspection involved 238 inspector hours onsite by five NRC inspectors. Of the 15 areas inspected, no apparent items of noncompliance or deviations were identified in nine areas, seven apparent items of noncompliance were identified in six areas (failure to provide basis for a special test procedure; failure to review and approve a procedure; failure to issue an official Safety-Related List; failure to utilize status reports; failure to adequately address Technical Specification requirement 4.6; failure to follow procedures relating to maintenance; and failure to update drawings).

REPORTS RECEIVED FROM LICENSEE

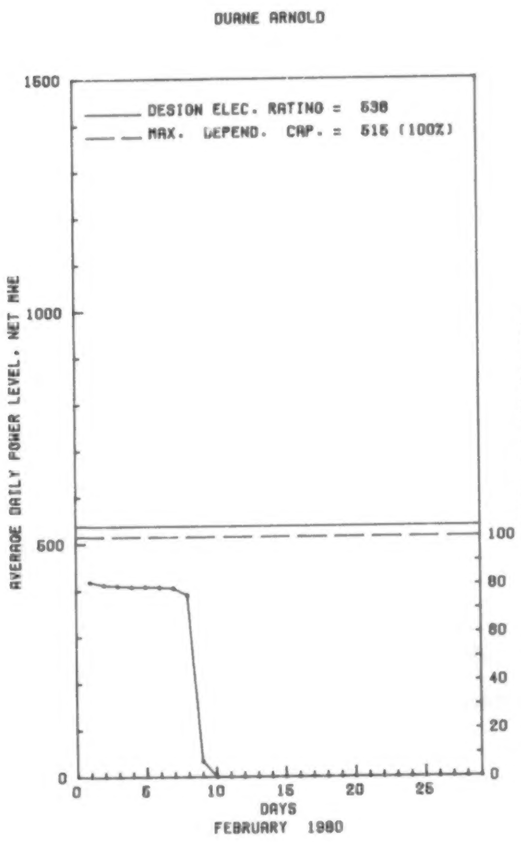
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-39/ 03L-0	12/31/79	01/30/80	30 Day LER	Engine low oil level limit was exceeded.

REVIEWED BY OMPA LOEB: John J. Riggs DATE: 4-10-80

REVIEWED BY NRR DOR: DATE: 4/14/80

DUANE ARNOLD

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-331
2. Utility Contact: J. Van Sickel (319) 851-5611
3. Licensed Thermal Power (MWt): 1658
4. Nameplate Rating (Gross MWe): 663 x 0.9 = 597
5. Design Electrical Rating (Net MWe): 538
6. Maximum Dependable Capacity (Gross MWe): 545
7. Maximum Dependable Capacity (Net MWe): 515
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	696.0	1,440.0	44,520.0
12. Hours Reactor Was Critical	200.1	944.1	31,508.2
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	200.1	944.1	30,763.2
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	243,768	1,219,968	38,349,528
17. Gross Electrical Energy (MWH)	84,693	625,336	12,852,702
18. Net Electrical Energy (MWH)	79,173	601,349	12,016,613
19. Unit Service Factor	28.8	65.6	69.1
20. Unit Availability Factor	28.8	65.6	69.1
21. Unit Capacity Factor (MDC Net)	22.1	56.1	52.4
22. Unit Capacity Factor (DER Net)	21.1	51.8	50.2
23. Unit Forced Outage Rate	.0	.0	20.4

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 04/10/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
1	02/09/80	S	495.9	C	3				PLANT WAS SHUTDOWN FOR REFUELING ON 02/09/80, WHILE SHUTTING DOWN AND AT APPROXIMATELY 6% POWER, A REACTOR SCRAM AND TURBINE TRIP OCCURRED DURING TURBINE CONTROL VALVE TESTING.

SUMMARY

THE UNIT WAS SHUTDOWN ON 02/09/80 FOR REFUELING, FOLLOWING A REACTOR SCRAM DURING TURBINE CONTROL VALVE TESTING.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

FARLEY 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Alabama/Houston 24 miles SE of Dothan
2.	Electric Reliability Council	SERC
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2652
5.	Design Electrical Rating (MWE Net)	829
6.	Date of Initial Criticality	August 9, 1977
7.	Date of Commercial Operation	December 1, 1977
8.	Condenser Cooling Method	Once Through With Forced Draft Cooling Towers
9.	Condenser Cooling Water Source	Chattahoochee River

Utility & Contractor Information

10.	Licensee	Alabama Power Company
11.	Corporate Address	600 North 18th Street Birmingham, Alabama
12.	Corporate Contact	F. L. Clayton, Jr. Senior Vice President (205-323-5341)
13.	Architect/Engineer	Bechtel/SSI
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Daniel

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. Lewis
18.	IE Senior Inspector	R. D. Martin
19.	IE Resident Inspector	W. H. Bradford (205) 899-3386
20.	Licensing Project Manager	E. A. Reeves
21.	Docket Number	50-348
22.	License Number/Date of Issue	NPF-2, 6/25/77
23.	Public Document Room	G. S. Houston Memorial Library 212 W. Burdeshaw Street Dothan, Alabama 36301

INSPECTION STATUS

ENFORCEMENT STATUS

AS REQUIRED BY TECHNICAL SPECIFICATION 6.8.1, WRITTEN PROCEDURES SHALL BE ESTABLISHED, IMPLEMENTED AND MAINTAINED. PLANT PROCEDURE FNP-1-AP-52, EQUIPMENT STATUS CONTROL AND MAINTENANCE AUTHORIZATIONS STATES THAT THE RELEASING INDIVIDUAL WILL REVIEW ITEMS 12-20 ON THE WORK REQUEST AND WILL SIGN THE "RELEASE FOR WORK" SECTION ONLY IF THE REQUEST IS COMPLETE AND ACCURATE. CONTRARY TO THE ABOVE, FNP WORK REQUEST 19749, WHICH CALLED FOR THE REPAIR OF WASTE GAS TANK NO. 8 INLET ISOLATION VALVE (Q1G-22V0858) IN ACCORDANCE WITH PLANT PROCEDURE FNP-0-MP-21.1 WAS RELEASED FOR WORK ALTHOUGH THE PLANT CONDITIONS NECESSARY TO PERFORM THE WORK WERE NOT IN AFFECT. THE TAGGING OPERATION ORDER NO 79-1671 REFERENCED ON THE WORK REQUEST DID NOT ESTABLISH PLANT CONDITIONS NECESSARY TO PERFORM THE WORK SPECIFIED ON THE WORK REQUEST.
(7942 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

- RETURNED TO POWER ON FEBRUARY 15, 1980 FOLLOWING AN OUTAGE TO MAKE MODIFICATIONS REQUIRED BY NUREG 0578.
- LAST IE SITE INSPECTION DATE: FEBRUARY 24-29, 1980
- INSPECTION REPORT NO: 50-348/80-07

SUMMARY

INSPECTION SUMMARY

- INSPECTION DECEMBER 10-14 (79-42): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 35 INSPECTOR-HOURS ON SITE IN THE AREAS OF THE RADIOACTIVE WASTE MANAGEMENT PROGRAM, INCLUDING RADIOACTIVE EFFLUENT RELEASE REPORTS, RADIOACTIVE EFFLUENT RELEASE RECORDS, PROCEDURES FOR CONTROLLING THE RELEASE OF EFFLUENTS; REVIEW OF LICENSEE ACTION ON IE BULLETINS AND CIRCULARS AND FOLLOWUP ON AN UNPLANNED GASEOUS RELEASE. OF THE SIX AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN FIVE AREAS; ONE APPARENT ITEM OF NONCOMPLIANCE WAS FOUND IN ONE AREA (INFRACTION - FAILURE TO FOLLOW PROCEDURES FOR REVIEWING MAINTENANCE WORK REQUEST).

INSPECTION JANUARY 30-31 (80-01): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 14 INSPECTOR-HOURS ON SITE IN THE AREAS AND ACTIVITIES RELATIVE TO IE BULLETIN 79-17 (PIPE CRACKS IN STAGNANT BORATED WATER SYSTEMS AT PWR PLANTS). NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

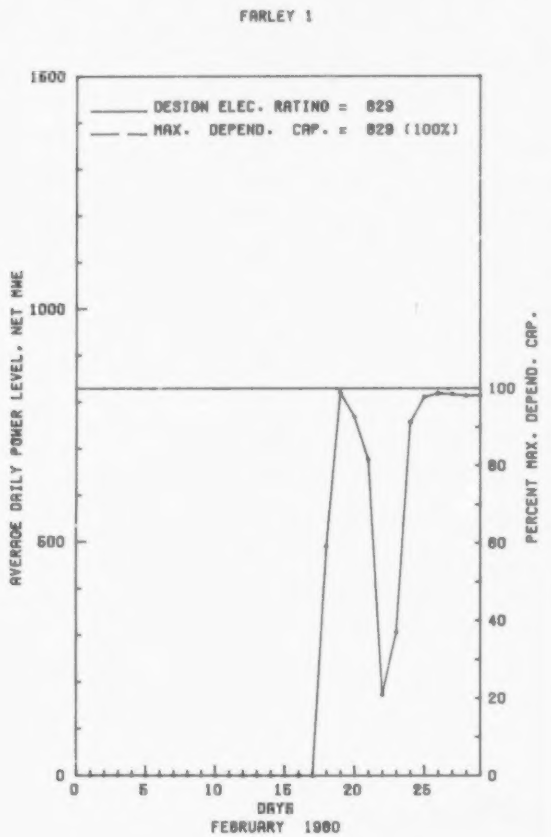
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-065/ 03L-0	11/22/79	01/11/80	30 DAY LER	MAIN STEAM ISO VALVE 3370C WAS INOPERABLE VIA HAND SWITCH; ALL 5 OTHERS OPERABLE. HAND SWITCH REPLACED AND CHECKED OUT SATISFACTORY. VALVE WOULD CLOSE ON AUTO ISOLATION.
79-067/ 03L-0	12/22/79	01/25/80	30 DAY LER	RIVER WATER B TRAIN PUMPS INOPERABLE DUE TO A FAULTY LEVEL.
79-068/ 03L-0	12/26/79	01/25/80	30 DAY LER	PERSONNEL ERROR RESULTED IN NO CHARGING PUMP ALIGNED TO "B".
80-001/ 03L-0	01/06/80	02/01/80	30 DAY LER	FUEL STORAGE POOL RAD MONITOR INOPERABLE.
80-002/ 03L-0	01/09/80	02/07/80	30 DAY LER	DIESEL GENERATOR INOPERABLE DUE TO LOW DAY TANK (FUEL) LEVEL.
80-003/ 03L-0	01/04/80	02/11/80	30 DAY LER	NEUTRON FLUX CHANNEL CALIBRATION NOT PERFORMED (HEAT BALANCE) WITHIN SURVEILLANCE PERIOD.
80-004/ 03L-0	01/09/80	02/07/80	30 DAY LER	RAD MONITOR INOPERABLE.

REVIEWED BY OMPA LOEB: T.C. CINTULA **DATE:** 4·10·80

REVIEWED BY NRR DOR: Ed A. Reeves **DATE:** 4-14-80

FARLEY 1

AVERAGE POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-348
2. Utility Contact: W. G. Hairston,III (205) 899-5156
3. Licensed Thermal Power (MWt): 2652
4. Nameplate Rating (Gross MWe): 1045 x 0.85 = 888
5. Design Electrical Rating (Net MWe): 829
6. Maximum Dependable Capacity (Gross MWe): 860
7. Maximum Dependable Capacity (Net MWe): 829
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	696.0	1,440.0	19,794.0
12. Hours Reactor Was Critical	314.6	1,020.0	11,955.5
13. Reactor Reserve Shutdown Hours	381.4	420.0	1,825.3
14. Hours Generator On-Line	267.3	963.5	11,554.0
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	639,186	2,360,605	28,958,619
17. Gross Electrical Energy (MWH)	205,704	763,174	9,287,212
18. Net Electrical Energy (MWH)	193,464	720,372	8,779,788
19. Unit Service Factor	38.4	66.9	58.6
20. Unit Availability Factor	38.4	66.9	58.6
21. Unit Capacity Factor (MDC Net)	33.5	60.3	53.7
22. Unit Capacity Factor (DER Net)	33.5	60.3	53.7
23. Unit Forced Outage Rate	18.5	10.0	7.8

24. Shutdowns Scheduled Over Next 6 Months (Type,Date,and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
011	01/31/80	S	368.1	D	4		ZZ	ZZZZZZ	CONTINUATION OF 01/31/80 SHUTDOWN TO PERFORM NUREG 0578 RELATED DESIGN CHANGES.
012	02/16/80	F	3.8	A	3		HH	INSTRU	UNSTABLE UNIT TRIPPED ON LO-LO STEAM GENERATOR LEVEL CAUSED BY FEED PUMP CONTROL. I & C REPLACED A DEFECTIVE CONTROL CARD IN 'B' SGFP CONTROL SYSTEM.
013	02/16/80	F	17.7	G	3		HH	ZZZZZZ	UNIT TRIPPED WHILE IN MODE 2 ON STEAM GENERATOR LO-LO LEVEL. THE REACTOR OPERATOR DID NOT MAINTAIN LEVEL WITH AUXILIARY FEEDWATER FLOW. REACTOR OPERATOR WAS REINSTRUCTED.
014	02/17/80	F	16.0	A	3		HH	INSTRU	UNIT TRIPPED ON LO-LO STEAM GENERATOR LEVEL CAUSED BY UNSTABLE FEEDPUMP CONTROL. I & C CORRECTED THE PROBLEM BY REVISING THE DELTA P PROGRAM IN THE FEEDPUMP SPEED CONTROL CIRCUIT.
015	02/18/80	F	7.8	B	5		HH	HTEXCH	UNIT WAS REDUCED AND HELD AT 50% REACTOR POWER TO REPAIR A LEAK ON THE SHELL SIDE OF 5B HEATER AND CHANGE OIL IN THE FEEDPUMP OIL CONDITIONERS WHICH CONTAINED WATER.
016	02/20/80	F	28.5	B	5		EB	INSTRU	UNIT WAS REDUCED AND HELD AT 86% REACTOR POWER TO INVESTIGATE MAIN GENERATOR EXCITER VOLTAGE REGULATOR OSCILLATIONS.
017	02/21/80	S	2.1	B	5		HA	VALVEX	UNIT WAS HELD AT 86% REACTOR POWER TO PERFORM MAIN TURBINE GOVERNOR CONTROL VALVE SURVEILLANCE TESTING.
018	02/22/80	F	14.6	H	3		HH	ZZZZZZ	UNIT TRIPPED DUE TO TRIP OF BOTH SGFP'S ON LOW SUCTION PRESSURE.
019	02/23/80	F	8.5	G	1		HH	ZZZZZZ	UNIT MANUALLY TRIPPED DUE TO TRIP OF BOTH SGFP'S ON LOW SUCTION PRESSURE.
020	02/23/80	F	16.1	B	5		HH	VALVEX	UNIT HELD AT 90% REACTOR POWER TO INVESTIGATE LOW SGFP SUCTION PRESSURE. ADJUSTED GLAND SEAL CONDENSER BYPASS VALVE AND A HEATER DRAIN TANK DUMP VALVE.

SUMMARY

THE UNIT REMAINED SHUTDOWN UNTIL 02/16/80 TO MAKE TMI-RELATED MODIFICATIONS. THREE UNIT TRIPS OCCURRED BETWEEN 02/16/80 AND 02/17/80 ON LO-LO STEAM GENERATOR LEVEL DURING STARTUP. UNIT TRIPS ALSO OCCURRED ON 02/22/80 AND 02/23/80 DUE TO TRIPS OF BOTH STEAM GENERATOR FEED PUMPS ON LOW SUCTION PRESSURE. THREE LOAD REDUCTIONS WERE NECESSARY FOR MAINTENANCE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

NRC POWER RESTRICTIONS: NONE

FITZPATRICK

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	New York/Oswego 8 miles NE of Oswego
2.	Electric Reliability Council	NPCC
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2436
5.	Design Electrical Rating (MWE Net)	821
6.	Date of Initial Criticality	November 17, 1974
7.	Date of Commercial Operation	July 28, 1975
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Lake Ontario

Utility & Contractor Information

10.	Licensee	Power Authority of the State of New York
11.	Corporate Address	10 Columbus Circle New York, New York 10019
12.	Corporate Contact	P. Lyon, Manager of Nuclear Operations
13.	Architect/Engineer	Stone & Webster
14.	Nuclear Steam Supply System	GE
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner (X246)
18.	IE Senior Inspector	E. McCabe (X251)
19.	IE Principal Inspector	T. F. Stetka (X276)
20.	Licensing Project Manager	P. Polk (X7872)
21.	Docket Number	50-333
22.	License Number/Date of Issue	DPR-59, 10/17/74
23.	Public Document Room	State University College of Oswego Penfield Library - Documents Oswego, NY 13126 (3[illegible]

INSPECTION STATUS

ENFORCEMENT STATUS

10 CFR 50 APPENDIX B CRITERION II STATES THE APPLICANT SHALL ESTABLISH A QUALITY ASSURANCE PROGRAM...THE PROGRAM SHALL PROVIDE FOR INDOCTRINATION AND TRAINING OF PERSONNEL...." THE LICENSEE'S QUALITY ASSURANCE PROGRAM STATES IN SECTION II SUBSECTION 2.8 "TRAINING AT THE PLANT SHALL BE PROVIDED FOR PERSONNEL...A TRAINING PROCEDURE SHALL BE ESTABLISHED WHICH INCLUDES PROVISIONS TO IMPLEMENT AN ADEQUATE TRAINING PROGRAM...." THE LICENSEE'S "INDOCTRINATION AND TRAINING PROCEDURE NO. 3 - GENERAL EMPLOYEE TRAINING REQUIRES IN SECTION 3.5.6 THAT "PLANT PERSONNEL QUALIFIED IN RADIATION PROTECTION SHALL BE PROVIDED WITH ANNUAL RETRAINING." CONTRARY TO THE ABOVE TWO INDIVIDUALS EXCEEDED THE ANNUAL RETRAINING INTERVAL BY SIX AND TEN MONTHS RESPECTIVELY AS OF DECEMBER 1979.
(7918 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

+ An approximate 6 inch pipe crack has developed on the suction line to condensate booster pump B. A temporary fix has been made. The offgas system recombiner is out of service which has resulted in higher than normal stack releases.

FACILITY ITEMS (PLANS AND PROCEDURES):

Refueling outage scheduled for April. 1980.

MANAGERIAL ITEMS:

+ Effective April 1, 1980 the positions of Resident Manager and Superintendent of Power will be assumed by new personnel.

PLANT STATUS:

+ The plant is operating at near full power with an average offgas rate at the SJAE of 11,000 microcuries per second and an average stack release rate of 1750 microcuries per second.

+ LAST IE SITE INSPECTION DATE: 3/14/80

+ INSPECTION REPORT NO: 50-333/80-03

SUMMARY

INSPECTION SUMMARY

+ 50-333/80-01 - Feb 4-8: Routine, unannounced inspection of plant operations, Licensee Event Reports (LERs), IE Bulletin responses, licensee action on IE Circulars, and licensee action on previous inspection findings. This report also documents in-office review of LERs and the 1977 10 CFR 50.59 Report. The inspection involved 48-hours onsite by one region based inspector. Two items of noncompliance were identified (failure to comply with Technical Specifications prior to making a control rod inoperable; uncontrolled release of liquid with pH outside of Technical Specification required range).

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-096/ 03L	10/28/79	11/27/79	30 Day LER	Reactor water chloride and conductivity increased due to tube leak on waste concentrator. Conductivity exceeded TS limit
79-097/ 03L	10/30/79	11/27/79	30 Day LER	EDG C could not force parallel during testing. Event initiated reactor shutdown due to other system out of service
79-098/ 03L	10/31/79	11/28/79	30 Day LER	Failure of radwaste building exhaust monitor B detected during surveillance testing
79-099/ 03L	11/05/79	12/04/79	30 Day LER	Hydraulic control unit for CR 18-35 was made inoperable to repair leaking access plug
79-100/ 03L	11/08/79	12/04/79	30 Day LER	HPCI was made inoperable to repair leaking gasket on hydraulic control valve seat drain
79-101/ 03L	11/08/79	12/04/79	30 Day LER	Six penetrations were found to be not sealed with fire barrier material. This condition has existed since plant construction
79-102/ 03L	11/08/79	12/05/79	30 Day LER	Reactor water conductivity exceeded TS limit due to condenser tube leaks
79-104/ 03L	11/10/79	12/10/79	30 Day LER	SRM A exhibited erratic operation during surveillance testing Problem cleared and cound not be recreated
79-105/ 03L	11/14/79	12/13/79	30 Day LER	Failure of EDG C lub oil recirculation pump
79-118/ 03L	12/31/79	01/28/80	30 Day LER	Drywell atmos monitor A failed while B monitor was out of service
80-001/ 03L	01/01/80	01/29/80	30 Day LER	APRM E downscale trip was found at 2/125 of scale vice TS limit 06 equal to or greater than 2.5/125 of scale
80-002/ 04X	01/02/80	02/12/80	30 Day LER	Makeup water system neutralizer tank discharge valve stuck open causing release of waste water with pH outside of TS limits
80-003/ 03L	01/06/80	01/29/80	30 Day LER	Torus water level decreased below TS limit during testing of torus to drywell vacuum breakers
80-004/ 03L	01/07/80	01/31/80	30 Day LER	Reactor low pressure switch for LPCI and CS permissive found out of spec during surveillance testing
80-005/ 03L	01/07/80	02/06/80	30 Day LER	Failed to take reactor water samples for chlorides, conductivity and iodine every four hours during plant startup
80-006/ 03L	01/08/80	02/06/80	30 Day LER	Management review of W/R indicated that core spray injection valve 14-MOV-12A did not operate properly during surveillance test

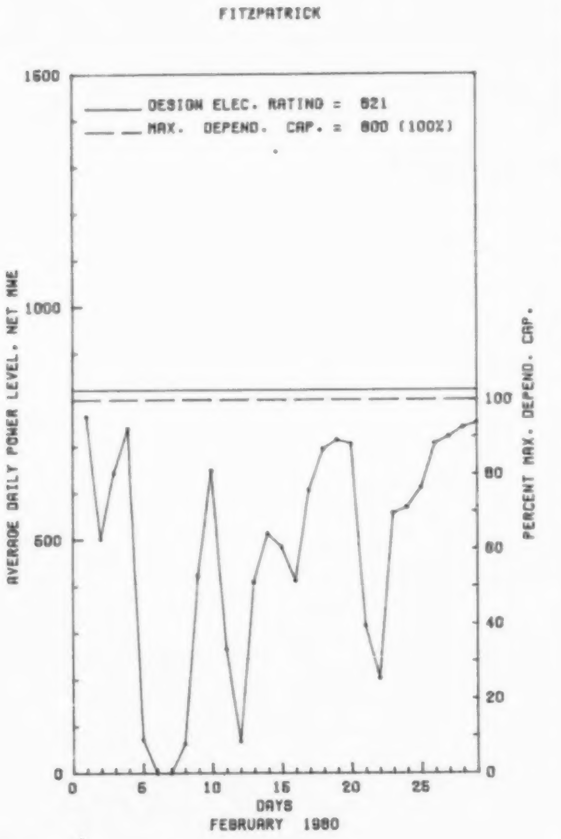
(SEE CONTINUATION PAGE F-5A)

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 4-10-80

REVIEWED BY NRR DOR: Philip J. Polk **DATE:** 4/11/80

FITZPATRICK

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

ITEM 21 CALCULATED BY USING A WEIGHTED AVERAGE

1. Docket Number: 50-333
2. Utility Contact: E. Zufelt (315) 342-3840
3. Licensed Thermal Power (MWt): 2436
4. Nameplate Rating (Gross MWe): 981 x 0.9 = 883
5. Design Electrical Rating (Net MWe): 821
6. Maximum Dependable Capacity (Gross MWe): 830
7. Maximum Dependable Capacity (Net MWe): 800
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	696.0	1,440.0	40,273.0
12. Hours Reactor Was Critical	609.5	1,348.5	27,978.[illegible]
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	564.5	1,308.5	27,056.5
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,088,312	2,682,552	53,870,746
17. Gross Electrical Energy (MWH)	345,030	922,920	18,276,626
18. Net Electrical Energy (MWH)	333,465	893,400	17,643,975
19. Unit Service Factor	81.1	90.9	67.2
20. Unit Availability Factor	81.1	90.9	67.2
*21. Unit Capacity Factor (MDC Net)	59.9	77.6	604
22. Unit Capacity Factor (DER Net)	58.4	75.6	53.5
23. Unit Forced Outage Rate	18.9	9.1	20.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): SNUBBER INSPEC. & REFUELING - APRIL 1980.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
2	02/02/80	S	.0	H	5				POWER REDUCED FOR ROD PATTERN CHANGE.
3	02/05/80	F	80.8	A	1	80-016	SD	VALVOP	MOISTURE LEAKAGE INTO VALVE OPERATOR CAUSED MOTOR AND CONTROL CIRCUIT FAILURE. REPAIRED MOISTURE LEAK, PROVIDED A DRAINAGE PATH AND REPLACED MOTOR.
4	02/11/80	F	30.6	G	3				OPERATOR PULLED FUSED ON UNDERVOLTAGE DEVICE FOR 10300 BUS.
5	02/21/80	F	20.1	G	3				DURING CALIBRATION OF MAIN STEAM HI-RADIATION MONITORS A VENDOR EMPLOYEE ACCIDENTALLY BUMPED A LEVEL SWITCH ON CRD SCRAM DISCHARGE VOLUME.

SUMMARY

REACTOR SCRAMS OCCURRED ON 02/11/80 WHEN AN OPERATOR PULLED FUSES ON THE UNDERVOLTAGE DEVICE FOR THE 10300 BUS, AND ON 02/21/80 WHEN AN EMPLOYEE ACCIDENTALLY BUMPED A LEVEL SWITCH ON THE CONTROL ROD DRIVE SCREW DISCHARGE VOLUME DURING CALIBRATION OF THE MAIN STEAM HI-RADIATION MONITORS. THE UNIT WAS OFF-LINE FROM 02/05/80 TO 02/08/80 TO PLACE A MOTOR FOLLOWING MOISTURE LEAKAGE INTO A VALVE OPERATOR. POWER WAS BRIEFLY REDUCED ON 02/02/80 FOR CONTROL ROD PATTERN CHARGE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
80-007/ 03L	01/09/80	02/08/80	30 Day LER	RHR loop B made inoperable due to pipe hanger failure
80-008/ 03L	01/09/80	02/08/80	30 Day LER	Core spray injection valve failed to open during surveillance testing due to torque and limit switch misadjustment
80-009/ 03L	01/10/80	02/08/80	30 Day LER	Diesel fire pump made inoperable to correct engine cooling system problem
80-010/ 03L	01/11/80	02/09/80	30 Day LER	Recirculation loop temperature instruments were found to be out of calibration due to personnel error in performing the previous calibration
80-011/ 03L	01/10/80	02/09/80	30 Day LER	Hudraulic snubber was found to be inoperative during surveillance testing. Snubber was replaced with a spare
80-012/ 03L	01/11/80	02/09/80	30 Day LER	Emergency diesel generator A tripped on low oil pressure during surveillance testing
80-013/ 03L	01/14/80	02/12/80	30 Day LER	Reactor lo-lo-lo level switch A did not operate properly during surveillance testing. Replaced with new switch
80-014/ 03L	01/16/80	02/12/80	30 Day LER	Torus water exceeded TS limits during HPCI testing. Level variance now permitted by TS during similar testing
80-015/ 03L	01/23/80	02/19/80	30 Day LER	Failure of LPCI independent power supply "A" inverter
80-016/ 03L	01/25/80	02/22/80	30 Day LER	Failure of drywell floor drain sump inboard isolation valve 20-MOV-82 due to a grounding situation caused by flooding the MOV terminal box
80-017/ 03L	01/30/80	02/25/80	30 Day LER	Drywell to torus differential pressure decreased to less than 1.7 PSID for about 1 hr. due to reactor building cooling

FORT CALHOUN

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1. Location (state/county) (nearest population center)	Nebraska/Washington 19 miles N of Omaha
2. Electric Reliability Council	MARCA
3. Type of Reactor	PWR
4. Power Level (MWT)	1420
5. Design Electrical Rating (MWE Net)	457
6. Date of Initial Criticality	August 6, 1973
7. Date of Commercial Operation	June 20, 1974
8. Condenser Cooling Method	Once Through
9. Condenser Cooling Water Source	Missouri River

Utility & Contractor Information

10. Licensee	W. C. Jones Division Manager – Production Operations Omaha Public Power District
11. Corporate Address	1623 Harney Street Omaha, Nebraska 68102
12. Corporate Contact	T. E. Short Assistant General Manager
13. Architect/Engineer	Gibbs & Hill, Inc.
14. Nuclear Steam Supply System	Combustion Engineering
15. Constructor	Gibbs & Hill, Durham & Richardson, Inc.

Regulatory Information

16. IE Region responsible	IV
17. IE Branch Chief	G. L. Madsen
18. IE Senior Inspector	T. F. Westerman
19. IE Principal Inspector	D. Kelley
20. Licensing Project Manager	P. B. Erickson
21. Docket Number	50-285
22. License Number/Date of Issue	DPR-40, 5/24/73
23. Public Document Room	W. Dale Clark Library 215 S. 15th Street Omaha, Nebraska 68102

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

(1) Concrete shield wall at equipment hatch not installed per FSAR figure 11.2.2 (76-16)

FACILITY ITEMS (PLANS AND PROCEDURES):

MANAGERIAL ITEMS:

(1) Safety Audit and Review Committee review of Facility License Changes. (76-16)

PLANT STATUS:

No significant information

LAST IE SITE INSPECTION DATE: JANUARY 28 - FEBRUARY 1, 1980 +

INSPECTION REPORT NO: 50-285/80-02 +

SUMMARY

INSPECTION SUMMARY

+ INSPECTION ON JANUARY 14-17, 1980 (80-01): ROUTINE, UNANNOUNCED INSPECTION OF THE LICENSEE'S (1) PLANT OPERATIONS; AND (2) PRE-REFUELING ACTIVITIES. THE INSPECTION CONSISTED OF TWENTY-FIVE (25) INSPECTOR HOURS ON-SITE BY ONE (1) NRC INSPECTOR. OF THE TWO (2) AREAS INSPECTED NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION ON JANUARY 28 - FEBRUARY 1, 1980 (80-02): SPECIAL, ANNOUNCED INSPECTION OF ACTIVITIES RELATED TO THE SECOND PERIODIC CONTAINMENT INTEGRATED LEAK RATE TEST. THE INSPECTION CONSISTED OF ONE-HUNDRED SIX INSPECTOR-HOURS ON SITE BY TWO NRC INSPECTORS. IN THE ONE AREA INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

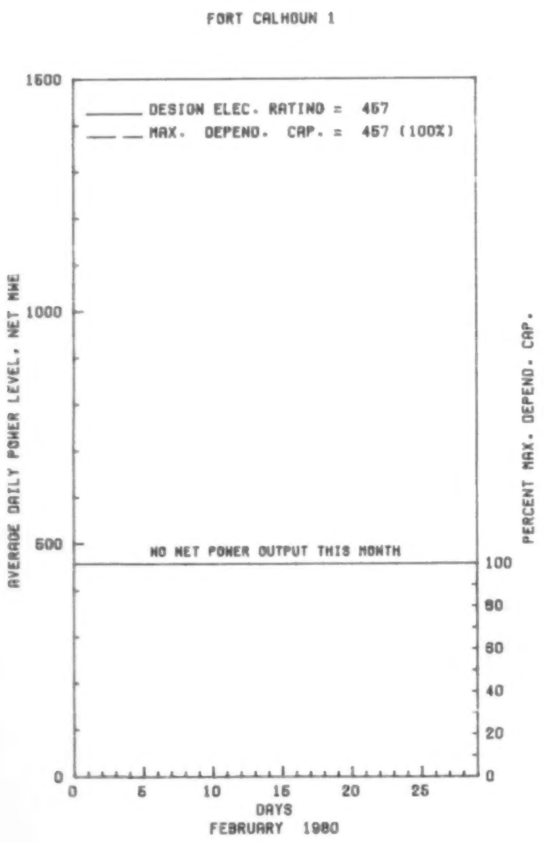
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-01 03-L0	1/2/80	1/7/80	COOLING WATER FAILURE TO CONTAINMENT COOLER WHILE TESTING REDUNDANT SYSTEM
80-02 01T-0	1/17/80	1/23/80	CONTAINMENT SUMP RECIR. VALVE FAILED TO CLOSE DURING SURVEILLANCE TEST

REVIEWED BY OMPA LOEB: MR Beebe **DATE:** 04/10/80

REVIEWED BY NRR DOR: PB Erickson **DATE:** 4/11/80

FORT CALHOUN

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-285
2. Utility Contact: B. J. Mickle (402) 536-4413
3. Licensed Thermal Power (MWt): 1420
4. Nameplate Rating (Gross MWe): 591 x 0.85 = 502
5. Design Electrical Rating (Net MWe): 457
6. Maximum Dependable Capacity (Gross MWe): 481
7. Maximum Dependable Capacity (Net MWe): 457
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

ITEM 21 CALCULATED BY USING A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	696.0	1,440.0	56,377.0
12. Hours Reactor Was Critical	.0	432.9	44,834.5
13. Reactor Reserve Shutdown Hours	.0	.0	1,309.5
14. Hours Generator On-Line	.0	430.5	43,881.6
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	574,731	53,809,758
17. Gross Electrical Energy (MWH)	0	190,684	17,850,834
18. Net Electrical Energy (MWH)	0	180,935	16,868,452
19. Unit Service Factor	.0	29.9	77.8
20. Unit Availability Factor	.0	29.9	77.3
21. Unit Capacity Factor (MDC Net)	.0	27.5	66.0
22. Unit Capacity Factor (DER Net)	.0	27.5	65.5
23. Unit Forced Outage Rate	.0	.0	4.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 03/15/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-01	01/18/80	S	696.0	C	4				ANNUAL REFUELING AND MAINTENANCE OUTAGE CONTINUES.

SUMMARY

THE UNIT REMAINED SHUTDOWN THROUGHOUT THE REPORT PERIOD FOR REFUELING.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B- Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

FORT ST. VRAIN

NRC POWER RESTRICTIONS: Restricted to 70% of Licensed Thermal Power.

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Colorado/Weld 35 miles N of Denver (near Platteville)
2.	Electric Reliability Council	WSCC
3.	Type of Reactor	HTGR
4.	Power Level (MWT)	842
5.	Design Electrical Rating (MWE Net)	330
6.	Date of Initial Criticality	January 31, 1974
7.	Date of Commercial Operation	July 1, 1979
8.	Condenser Cooling Method	Towers
9.	Condenser Cooling Water Source	South Platte River/Shallow Wells

Utility & Contractor Information

10.	Licensee	Public Service Company of Colorado
11.	Corporate Address	P. O. Box 840 Denver, Colorado 80201
12.	Corporate Contact	C. K. Millen Senior Vice President
13.	Architect/Engineer	Sargent & Lundy
14.	Nuclear Steam Supply System	General Atomic Company
15.	Constructor	General Atomic Company

Regulatory Information

16.	IE Region responsible	IV
17.	IE Branch Chief	G. L. Madsen
18.	IE Senior Inspector	T. F. Westerman
19.	IE Resident Inspector	M. W. Dickerson (303) 785-2281
20.	Licensing Project Manager	G. Kuzmycz
21.	Docket Number	50-267
22.	License Number/Date of Issue	DPR-34, 12/21/73
23.	Public Document Room	Greeley Public Library City Complex Building Greeley, Colorado 80631

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO TECHNICAL SPECIFICATION 4.6.1, LIMITING CONDITION FOR OPERATION, BOTH DIESEL GENERATOR DAY TANKS CONTAINED LESS THAN 500 GALLONS WHILE REACTOR OPERATED AT POWER.
(8001 INFRACTION)

CONTRARY TO GENERALLY ACCEPTED PRACTICES IN THE INDUSTRY, A DIGITAL VOLTMETER WAS NOT WARMED UP AS REQUIRED BY THE MANUFACTURER'S TECHNICAL MANUAL AND ITS ACCURACY WAS THEREFORE QUESTIONABLE.
(8001 DEVIATION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

Areas related to potential equipment modification, which are under review and discussion with the NRC include: helium circulator auxiliaries; accident analysis relating to outlet temperature dispersion and orifice settings; hangers for 2" and smaller piping; and plant oscillations.

FACILITY ITEMS (PLANS AND PROCEDURES):

Implementation of 10 CFR 73.55

MANAGERIAL ITEMS:

PLANT STATUS:

THE REACTOR WAS SHUT DOWN ON 10/26/79 FOR THE REPLACEMENT OF 1B HELIUM CIRCULATOR WHICH DEVELOPED A LEAK IN THE STATIC SEAL ACTUATING MECHANISM. THE FACILITY IS EXPECTED TO BE STARTED UP IN EARLY MARCH. CIRCULATOR REPLACEMENT IS COMPLETE. THE PLANT IS WAITING FOR PARTS TO REPAIR THE ACM DIESEL GENERATOR.

LAST IE SITE INSPECTION DATE: DECEMBER 31, 1979 - JANUARY 31, 1980

INSPECTION REPORT NO: 50-267/80-01

SUMMARY

INSPECTION SUMMARY

INSPECTION DECEMBER 31, 1979 - JANUARY 31, 1979 (80-01): ROUTINE, ANNOUNCED INSPECTION OF REVIEW OF EVENT REPORTS; CALIBRATION; SURVEILLANCE; PART 21 REPORT; MAINTENANCE; PHYSICAL SECURITY; PLANT OPERATIONS; ONSITE REVIEW COMMITTEE; OFFSITE REVIEW COMMITTEE; REVIEW AND AUDITS; AND REPORT REVIEWS. WITHIN THE TWELVE (12) AREAS INSPECTED, ONE ITEM OF NONCOMPLIANCE WAS IDENTIFIED (INFRACTION - IMPROPER DESIGN) AND ONE DEVIATION FROM GENERALLY ACCEPTED PRACTICES IN INDUSTRY.

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-02 03-L0	1/7/80	2/5/80	STARTUP CHANNEL SCRAM PROTECTION @ LESS THAN 10E5 CPS NOT OPERABLE
80-03 03-L0	1/10/80	2/7/80	BACKUP BEARING WATER FROM ACCUMULATOR TO LOOP 2 HE CIRCULATORS IMPAIRED DUE TO THREE IMPAIRED HANGERS.
80-04 03-L0	1/14/80	2/12/80	SNUBBER HOS-29 AND HRS-357 DETERMINED TO BE INOPERABLE
80-05 03-L0	1/11/80	2/8/80	GASEOUS RELEASE MADE WITH RECORDER INOPERABLE

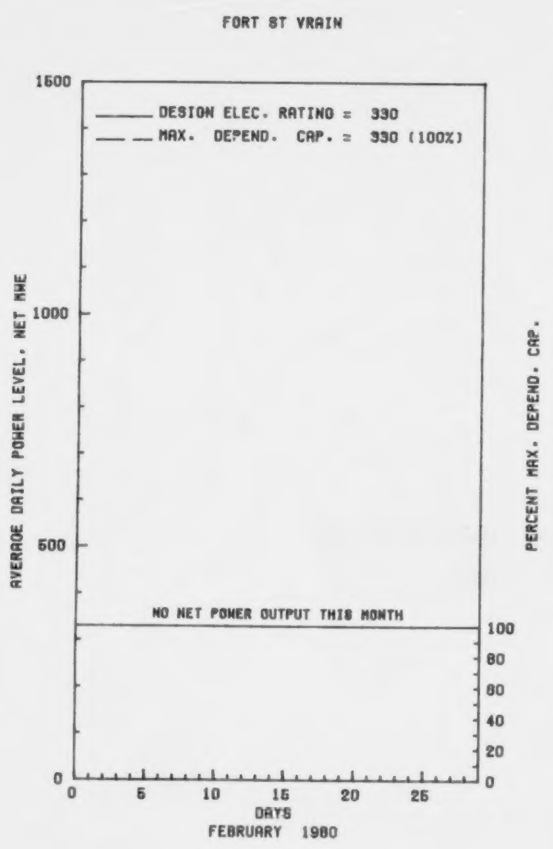
REVIEWED BY OMPA LOEB: T. C. Cintula **DATE:** 4·10·80

REVIEWED BY NRR DOR: S. Kuzmycz **DATE:** IV. 11. 1980

best copy available

FORT ST. VRAIN

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-267
2. Utility Contact: J. W. GAHM (303) 785-2253
3. Licensed Thermal Power (MWt): 842
4. Nameplate Rating (Gross MWe): 403 x 0.85 = 343
5. Design Electrical Rating (Net MWe): 330
6. Maximum Dependable Capacity (Gross MWe): 342
7. Maximum Dependable Capacity (Net MWe): 330
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): 231
10. Reasons for Restrictions, If Any: NRC RESTRICT. PEND. RESOL. OF TEMP. FLUCTUATIONS

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	696.0	1,440.0	5,857.0
12. Hours Reactor Was Critical	151.5	187.5	2,720.3
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	.0	.0	982.2
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	2,886	3,292	481,247
17. Gross Electrical Energy (MWH)	0	0	140,796
18. Net Electrical Energy (MWH)	0	0	123,584
19. Unit Service Factor	.0	.0	16.8
20. Unit Availability Factor	.0	.0	16.8
21. Unit Capacity Factor (MDC Net)	.0	.0	6.4
22. Unit Capacity Factor (DER Net)	.0	.0	6.4
23. Unit Forced Outage Rate	.0	.0	65.1

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 03/10/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
79-14	10/26/79	S	696.0	B	4	80-001			PLANT MAINTENANCE & INSTALLATION OF REGION CONSTRAINT DEVICES COMPLETED DECEMBER 24, 1979. PLANT REMAINED SHUTDOWN FOR REPLACEMENT OF ONE CIRCULATOR DUE TO A RUPTURED STATIC SEAL (REFERENCE LER 80-001/01-T-0). REPLACEMENT COMPLETED & REACTOR OPERATED AT LOW POWER LEVELS DURING THE PERIOD FROM FEBRUARY 16, 1980. GENERATOR REMAINED OFF-LINE.

SUMMARY

THE UNIT REMAINED SHUTDOWN THROUGHOUT THE REPORT PERIOD FOR REPLACEMENT OF A CIRCULATOR DUE TO A RUPTURED STATIC SEAL.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

GINNA

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1. Location (state/countv) (nearest population center)	New York/Wayne 15 miles NE of Rochester
2. Electric Reliability Council	NPCC
3. Type of Reactor	PWR
4. Power Level (MWT)	1520
5. Design Electrical Rating (MWE Net)	470
6. Date of Initial Criticality	November 8, 1969
7. Date of Commercial Operation	July 1970
8. Condenser Cooling Method	Once Through
9. Condenser Cooling Water Source	Lake Ontario

Utility & Contractor Information

10. Licensee	Rochester Gas & Electric Corporation
11. Corporate Address	89 East Avenue Rochester, New York
12. Corporate Contact	Leon D. White, Jr., Vice President Electric and Steam Production
13. Architect/Engineer	Gilbert
14. Nuclear Steam Supply System	Westinghouse
15. Constructor	Bechtel

Regulatory Information

16. IE Region responsible	I
17. IE Branch Chief	E. J. Brunner
18. IE Senior Inspector	R. Keimig
19. IE Principal Inspector	R. S. Markowski
20. Licensing Project Manager	J. J. Shea
21. Docket Number	50-244
22. License Number/Date of Issue	DPR-18, 9/19/69
23. Public Document Room	Rochester Public Library Business and Social Science Division 115 South Avenue Rochester, New York 14604

INSPECTION STATUS

ENFORCEMENT STATUS

10 CFR 71.3, "REQUIREMENT FOR LICENSE," REQUIRES THAT NO LICENSEE SUBJECT TO THE REGULATIONS IN THIS PART SHALL (A) DELIVER ANY LICENSED MATERIALS TO A CARRIER FOR TRANSPORT OR (B) TRANSPORT LICENSED MATERIAL EXCEPT AS AUTHORIZED IN A GENERAL LICENSE OR SPECIFIC LICENSE ISSUED BY THE COMMISSION, OR AS EXEMPTED IN THIS PART. CONTRARY TO THE ABOVE, ON JUNE 26, 1978, FEBRUARY 1, 1979, AND MAY 29, 1979, THE LICENSEE DELIVERED: 262 CI, 86 CI, AND 19.88 CI, RESPECTIVELY, OF LICENSED MATERIAL TO A CARRIER FOR TRANSPORT WITHOUT AUTHORIZATION IN A GENERAL OR A SPECIFIC LICENSE AND NO EXEMPTION TO 10 CFR 71 WAS APPLICABLE.
(7918 INFRACTION)

10 CFR 71.5, "TRANSPORTATION OF LICENSED MATERIAL," REQUIRES THAT NO LICENSEE DELIVER ANY LICENSED MATERIAL TO A CARRIER FOR TRANSPORT, UNLESS THE LICENSEE COMPLIES WITH THE APPLICABLE REQUIREMENTS OF THE DEPARTMENT OF TRANSPORTATION REGULATIONS 49 CFR PARTS 170-189. CONTRARY TO THE ABOVE, ON FEBRUARY 1, 1979, THE LICENSEE DELIVERED 86 CI OF LICENSED MATERIAL TO A CARRIER FOR TRANSPORT WITHOUT IDENTIFYING THE PRESENCE OF 58 MILLICURIES OF STRONTIUM 90 AS REQUIRED BY REGULATION 49 CFR PART 172.203(D).
(7918 INFRACTION)

10 CFR 71, APPENDIX E, CRITERION 16, STATES IN PART: "MEASURES SHALL BE ESTABLISHED TO ASSURE THAT CONDITIONS ADVERSE TO QUALITY...ARE...PROMPTLY CORRECTED." PROCEDURE A-1801, GINNA STATION RESPONSE TO INTERNAL AUDITS, REVISION 0, STEPS 3.3.1, 3.3.3 AND 3.3.5 STATE IN PART THAT INDIVIDUAL AUDIT FINDINGS ARE PRESENTED ON AUDIT FINDING CORRECTIVE ACTION REPORTS (AFCAR). THE RESPONSE TO THE AUDIT FINDING IS FORMULATED AND INCLUDES THE PROPOSED CORRECTIVE ACTION AND A TARGET COMPLETION DATE, AND THE RESPONSE IS SUBMITTED TO THE AUDIT TEAM LEADER ON OR BEFORE THE DUE DATE. CONTRARY TO THE ABOVE, AFCAR-1 OF AUDIT REPORT NO. 79-33:CA SPECIFIED A RESPONSE DUE DATE OF NOVEMBER 12, 1979; AND, AS OF DECEMBER 5, 1979 NO RESPONSE HAD BEEN FORMULATED.
(7918 INFRACTION)

10 CFR 71, APPENDIX E, "QUALITY ASSURANCE CRITERIA FOR SHIPPING PACKAGES OF RADIOACTIVE MATERIAL," CRITERION 5, "INSTRUCTIONS, PROCEDURES AND DRAWINGS" REQUIRES THAT ACTIVITIES AFFECTING QUALITY BE PRESCRIBED BY DOCUMENTED INSTRUCTIONS, PROCEDURES OR DRAWINGS OF A TYPE APPROPRIATE TO THE CIRCUMSTANCES AND THAT THESE ACTIVITIES BE ACCOMPLISHED IN ACCORDANCE WITH THESE INSTRUCTIONS, PROCEDURES OR DRAWINGS. CONTRARY TO THE ABOVE, ON FEBRUARY 1, 1979 AND MAY 29, 1979 THE LICENSEE DELIVERED 86 CI AND 19.88 CI OF LICENSED MATERIAL TO A CARRIER FOR TRANSPORT IN A PACKAGE WHICH WAS LOADED AND CLOSED WITHOUT WRITTEN PROCEDURES.
(7918 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

FACILITY ITEMS (PLANS AND PROCEDURES):

+ Unit operated at full power.

PLANT STATUS:

+ Routine power operation.

+ LAST IE SITE INSPECTION DATE: 3/6/80

+ INSPECTION REPORT NO: 50-244/80-04

SUMMARY

INSPECTION SUMMARY

+ 50-244/80-01 - Jan 15-17: Routine, unannounced inspection by two region-based inspectors (34 hr. onsite) of plant operations including: operating records; plant tours; compliance with Technical Specifications; action on previous inspection findings; and, followup review of licensee action on IE Circulars. No items of noncompliance were identified.

REPORTS RECEIVED FROM LICENSEE

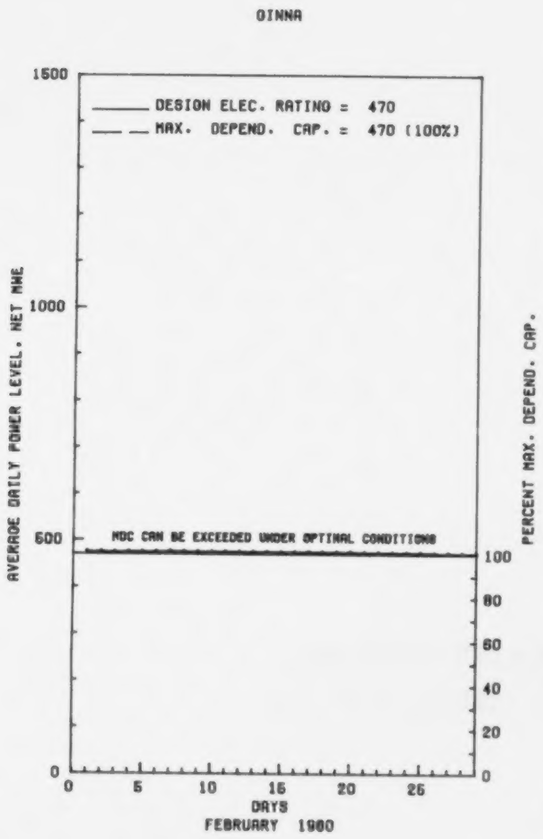
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
80-001/03L	01/18/80	02/15/80	30 Day LER	1A Emergency Diesel found inoperable during testing due to improper governor speed setting

REVIEWED BY OMPA LOEB: T. C. Cintula **DATE:** 4.10.80

REVIEWED BY NRR DOR: ______ **DATE:** ______

GINNA

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-244
2. Utility Contact: A. E. McNamara (716) 546-2700 x291-205
3. Licensed Thermal Power (MWt): 1520
4. Nameplate Rating (Gross MWe): 608 x 0.85 = 517
5. Design Electrical Rating (Net MWe): 470
6. Maximum Dependable Capacity (Gross MWe): 490
7. Maximum Dependable Capacity (Net MWe): 470
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

CAPACITY FACTORS CALCULATED WITH A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	696.0	1,440.0	89,952.0
12. Hours Reactor Was Critical	696.0	1,440.0	69,119.8
13. Reactor Reserve Shutdown Hours	.0	.0	1,621.8
14. Hours Generator On-Line	696.0	1,440.0	67,396.3
15. Unit Reserve Shutdown Hours	.0	.0	8.5
16. Gross Thermal Energy (MWH)	1,055,352	2,182,920	90,458,722
17. Gross Electrical Energy (MWH)	346,869	718,938	29,404,721
18. Net Electrical Energy (MWH)	330,263	684,461	27,846,009
19. Unit Service Factor	100.0	100.0	74.9
20. Unit Availability Factor	100.0	100.0	74.9
21. Unit Capacity Factor (MDC Net)	101.0	101.1	68.3
22. Unit Capacity Factor (DER Net)	101.0	101.1	68.3
23. Unit Forced Outage Rate	.0	.0	9.7

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUEL. & MAINT. SCHEDULED FOR 03/28/80.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
NONE									

SUMMARY

OPERATION WAS AT NEAR FULL POWER DURING THE REPORT PERIOD, WITH THREE MINOR REDUCTIONS (98%) FOR PERIODIC TESTING OF THE AUXILIARY FEEDWATER SYSTEM. ON 02/11/80, LOAD WAS REDUCED TO 95% DURING AN AUTOMATIC TURBINE RUNBACK.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

HADDAM NECK

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Connecticut/Middlesex 13 miles E of Meriden
2.	Electric Reliability Council	NPCC
3.	Type of Reactor	PWR
4.	Power Level (MWT)	1825
5.	Design Electrical Rating (MWE Net)	575
6.	Date of Initial Criticality	July 24, 1967
7.	Date of Commercial Operation	January 1, 1968
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Connecticut River

Utility & Contractor Information

10.	Licensee	Connecticut Yankee Atomic Power Co.
11.	Corporate Address	P. O. Box 270 Hartford, Connecticut
12.	Corporate Contact	D. Switzer, President
13.	Architect/Engineer	Stone & Webster
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Senior Inspector	R. Keimig
19.	IE Principal Inspector	W. J. Lazarus
20.	Licensing Project Manager	R. D. Silver
21.	Docket Number	50-213
22.	License Number/Date of Issue	DPR-61, 12/27/74
23.	Public Document Room	Russell Library 119 Broad Street Middletown, Connecticut 06457

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

Nothing significant to report.

- LAST IE SITE INSPECTION DATE: 2/20-22/80
- INSPECTION REPORT NO: 50-213/80-03

SUMMARY

INSPECTION SUMMARY

- 50-213/79-20 - Nov 27-30 & Dec 3-7: Routine, unannounced inspection by a regional-based inspector and one HQ based NRR engineer (52 hours) of the Quality Assurance Program (QAP) implementation in the areas of QA/QC administration; design changes/modification program; design changes and modifications; document control program; QAP changes; and followup on previously identified items. Noncompliances: None.
- 50-213/79-21 - Dec 17-20: Special, unannounced inspection conducted by inspectors dispatched to the site in response to a release of radioactive materials through the plant vent stack on December 16, 1979. The inspection involved 81 hours onsite by three NRC inspectors. Noncompliances: One (Environmental Technical Specification 2.4.3.1 - release limit was exceeded).
- 50-213/80-01 - Jan 8-11: Routine, unannounced inspection by two regional-based inspectors (34 hours) of: Security Organization; Physical Barriers (Protected Area); Physical Barriers (Vital area); Lighting; Access Control (Identification, Authorization, Badging); Access Control (Vital Areas); Communications; and, Compensatory Measures. The inspection was initiated during back-shift hours. Noncompliances: 3 (Access Controls (Vital Area) - A vital area door was found unlocked and unattended; Compensatory Measures - Failure to maintain surveillance of protected area barriers and an exterior vital area door; and Communications - Failure to equip fixed security patrols with portable radios).
- 50-213/80-02 - Jan 22-25: Routine, announced inspection by two regional-based inspectors (36 hours) to discuss details related to establishment of an on-site office for a full-time NRC inspector at the facility, to turnover the assignment of the principal IE inspector and review of plant operations. Noncompliances: None.

REPORTS RECEIVED FROM LICENSEE

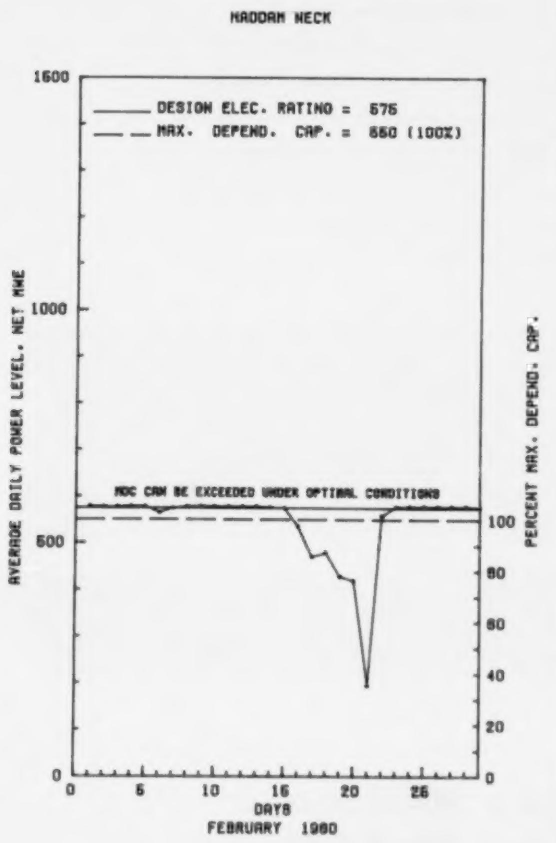
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
NONE				

REVIEWED BY OMPA LOEB: T. C. Cintula **DATE:** 4·10·80

REVIEWED BY NRR DOR: Ralph Caruso **DATE:** 4/14/80

HADDAM NECK

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-213
2. Utility Contact: C. Kokoszka (203) 267-2556
3. Licensed Thermal Power (MWt): 1825
4. Nameplate Rating (Gross MWe): 667 x 0.9 = 600
5. Design Electrical Rating (Net MWe): 575
6. Maximum Dependable Capacity (Gross MWe): 577
7. Maximum Dependable Capacity (Net MWe): 550
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

CAPACITY FACTORS CALCULATED WITH A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	696.0	1,440.0	106,632.0
12. Hours Reactor Was Critical	696.0	1,440.0	92,636.7
13. Reactor Reserve Shutdown Hours	.0	.0	1,152.6
14. Hours Generator On-Line	696.0	1,440.0	88,380.2
15. Unit Reserve Shutdown Hours	.0	.0	369.9
16. Gross Thermal Energy (MWH)	1,200,939	2,557,032	152,698,186
17. Gross Electrical Energy (MWH)	397,210	847,431	58,201,897
18. Net Electrical Energy (MWH)	378,691	808,585	47,763,348
19. Unit Service Factor	100.0	100.0	82.9
20. Unit Availability Factor	100.0	100.0	83.2
21. Unit Capacity Factor (MDC Net)	98.9	102.1	82.8
22. Unit Capacity Factor (DER Net)	94.6	97.7	77.1
23. Unit Forced Outage Rate	.0	.0	7.2

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUELING, MAY 1980, APPROX. 8 TO 12 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
11	02/21/80	F	.0	H	5		ZZ	ZZZZZZ	STEAM GENERATOR CHEMISTRY OUT OF SPECIFICATION.

SUMMARY

OPERATION WAS AT NEAR FULL POWER DURING THE REPORT PERIOD, WITH A LOAD REDUCTION ON 02/21/80 TO BRING STEAM GENERATOR CHEMISTRY WITHIN SPECIFICATIONS. ON 02/16/80, THE "A" CIRCULATING WATER PUMP WAS REMOVED FROM SERVICE DUE TO MOTOR GROUND. OTHER BRIEF REDUCTIONS OCCURRED FOR TESTING AND MAINTENANCE.

LEGEND

[1] Type:
F–Forced
S–Scheduled

[2] Reason:
A–Equipment Failure
B–Maintenance or Test
C–Refueling
D–Regulatory Restriction
E–Operator Training & License Examination
F–Administrative
G–Operational Error
H–Other

[3] Method:
1–Manual
2–Manual Scram
3–Automatic Scram
4–Continuations
5–Load Reduction
9–Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

HATCH 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1. Location (state/county) (nearest population center)	Georgia/Appling 11 miles N of Baxley
2. Electric Reliability Council	Member of SE Electric Reliability Council
3. Type of Reactor	BWR
4. Power Level (MWT)	2436
5. Design Electrical Rating (MWE Net)	777
6. Date of Initial Criticality	September 12, 1974
7. Date of Commercial Operation	December 31, 1975
8. Condenser Cooling Method	Once Through With Forced Draft Cooling Towers
9. Condenser Cooling Water Source	Altamaha River

Utility & Contractor Information

10. Licensee	Georgia Power Company
11. Corporate Address	270 Peachtree Street, NE Atlanta, Georgia 30303
12. Corporate Contact	J. H. Miller Executive Vice President
13. Architect/Engineer	Southern Services, Inc.
14. Nuclear Steam Supply System	GE
15. Constructor	Georgia Power Company

Regulatory Information

16. IE Region responsible	II
17. IE Branch Chief	R. C. Lewis
18. IE Senior Inspector	H. C. Dance
19. IE Resident Inspector	R. F. Rogers (912) 367-9881
20. Licensing Project Manager	J. N. Hannon
21. Docket Number	50-321
22. License Number/Date of Issue	DPR-57, 8/6/74
23. Public Document Room	Appling County Public Library Parker Street Baxley, Georgia 31513

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO TECHNICAL SPECIFICATION 6.8.1.C, THE FUEL LEVELS WERE NOT RECORDED ON THE DATA SHEET FOR FIRE PUMPS 2 AND 3, ON FEBRUARY 15, 1979, JUNE 26, 1979, JULY 29, 1979, AUGUST 16, 1979, SEPTEMBER 17, 1979 AND OCTOBER 8, 1979.
(8001 DEFICIENCY)

SAFETY RELATED CABINETS AT 130 FOOT CORRIDOR OF THE CONTROL BUILDING ARE NOT PROTECTED AGAINST DISCHARGE OF WATER FROM THE SPRINKLER SYSTEM AS STATED BY SECTION IV.C.1.4.(G) OF THE EVALUATION OF FIRE PROTECTION PROGRAM.
(8002 DEVIATION)

DOCUMENTATION IS NOT PROVIDED TO DEMONSTRATE THAT THE FIRE PROTECTION MODIFICATIONS WERE ACCOMPLISHED UNDER THE CONTROL OF THE QA ORGANIZATION AS STATED BY SECTION IV.B.7 OF THE LICENSEE'S EVALUATION OF FIRE PROTECTION PROGRAM.
(8002 DEVIATION)

COMBUSTIBLE MATERIALS ARE STORED IN EAST CABLEWAY WHICH IS CONTRARY TO SECTION III.C.4.(B) OF THE LICENSEE'S EVALUATION OF FIRE PROTECTION PROGRAM.
(8002 DEVIATION)

HEAT DETECTORS ARE INSTALLED WITHIN THE CONTROL ROOM KITCHEN IN LIEU OF THE SMOKE DETECTORS STATED BY SECTION III.C.1.1 OF THE LICENSEE'S EVALUATION OF FIRE PROTECTION PROGRAM.
(8002 DEVIATION)

FLOORS ARE NOT MARKED IN THE RIVER INTAKE STRUCTURE TO PROHIBIT THE STORAGE OF COMBUSTIBLES AS STATED BY SECTION III.C.4 OF THE LICENSEE'S EVALUATION OF FIRE PROTECTION PROGRAM.
(8002 DEVIATION)

THE FIRE DOORS IN THE FIRE BARRIERS ENCLOSING THE CONTROL ROOM DO NOT FULLY MEET THE PROVISIONS OF NFPA-80 AS STATED BY THE LICENSEE'S EVALUATION OF FIRE PROTECTION PROGRAM SECTION III.C.1.2.
(8002 DEVIATION)

CONTROL VALVES ON OIL DRAIN PIPING SYSTEM FROM REACTOR RECIRCULATION M-G SET ROOMS WERE NOT LOCKED CLOSED AS STATED BY TABLE 3, ITEM NO. III.43 OF THE LICENSEE'S EVALUATION OF FIRE PROTECTION PROGRAM.
(8002 DEVIATION)

AUTOMATIC SPRINKLER SYSTEM INSTALLATIONS DO NOT FULLY MEET THE REQUIREMENTS OF NFPA-13 AS STATED BY SECTION IV.C.3.C OF THE LICENSEE'S EVALUATION OF FIRE PROTECTION PROGRAM.
(8002 DEVIATION)

MOBILE PLATFORM FOR FIRE FIGHTING OPERATIONS IS NOT PROVIDED IN THE NORTHWEST CABLE AREA AS STATED BY TABLE 3, ITEM III.39 OF THE LICENSEE'S EVALUATION OF FIRE PROTECTION PROGRAM.
(8002 DEVIATION)

FIRE BARRIER HAS NOT BEEN PROVIDED BETWEEN THE NO. 2 AND NO. 3 FIRE PUMP UNITS AS STATED BY SECTION IV.C.2.C OF THE LICENSEE'S EVALUATION OF FIRE PROTECTION PROGRAM.
(8002 DEVIATION)

THE AIR COMPRESSOR FOR REFILLING THE SELF CONTAINED BREATHING APPARATUS CYLINDERS IS LOCATED IN THE WATER TREATMENT BUILDING IN LIEU OF THE DIESEL GENERATOR BUILDING AND IS NOT PROVIDED WITH AN EVAILABLE POWER SUPPLY IN THE EVENT OF LOSS OF OFF SITE POWER AS STATED BY SECTION IV.B.4.H OF THE LICENSEE'S EVALUATION OF THE FIRE PROTECTION PROGRAM.
(8002 DEVIATION)

THE UNDERGROUND SUPPLY MAIN TO THE SPRINKLER/STANDPIPE SYSTEMS WITHIN THE RIVER INTAKE STRUCTURE DOES NOT MEET THE PROVISIONS OF NFPA-24 AS STATED BY SECTION IV.C.2.A OF THE LICENSEE'S EVALUATION OF FIRE PROTECTION PROGRAM.
(8002 DEVIATION)

A NUMBER OF FIRE PROTECTION CONTROL VALVES WERE NOT PROVIDED WITH THE CORRECT TYPE OF VALVES SUPERVISION AS STATED BY SECTION IV.C.3.B OF THE LICENSEE'S EVALUATION OF FIRE PROTECTION PROGRAM.
(8002 DEVIATION)

REPORTS RECEIVED FPOM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-102/ 03L-0	12/26/79	01/17/80	30 DAY LER	HPCI STEAM LINE HIGH FLOW SWITCH DRIFT.
79-103/ 03L-0	12/27/79	01/17/80	30 DAY LER	RCIC HIGH FLOW SWITCH DRIFT.
80-002/ 03L-0	01/12/80	01/31/80	30 DAY LER	STANDBY GAS TREATMENT VENTILATION FLOW LOW.
80-003/ 03L-0	01/05/80	01/24/80	30 DAY LER	RESIDUAL HEAT REMOVAL ISOLATION VALVE OPERATING TIME EXCEEDED.
80-004/ 03L-0	01/06/80	01/24/80	30 DAY LER	MAIN STEAM DRAIN ISOLATION VALVE FAILURE TO OPERATE.
80-006/ 03L-0	01/13/80	01/24/80	30 DAY LER	STANDBY GAS TREATMENT OPERABILITY VERIFICATION TIME LIMIT EXCEEDED.
80-007/ 03L-0	01/14/80	02/04/80	30 DAY LER	STANDBY SERVICE WATER PUMP DID NOT MEET OPERABILITY SPECIFICATION.
80-008/ 03L-0	01/15/80	01/30/80	30 DAY LER	SECONDARY CONTAINMENT AIRLOCK DOORS OPENED SIMULTANEOUSLY.
80-009/ 03L-0	01/18/80	01/31/80	30 DAY LER	DRYWELL/TORUS TEMPERATURE RECORDER ERRATIC.
80-012/ 03L-0	01/22/80	02/05/80	30 DAY LER	HPCI AND RCIC INOPERABLE.

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

PLANT OPERATING AT FULL POWER.

* LAST IE SITE INSPECTION DATE: FEBRUARY 2-29, 1980

* INSPECTION REPORT NO: 50-321/80-11

SUMMARY

INSPECTION SUMMARY

* INSPECTION JANUARY 8-11 (80-01): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 13 INSPECTOR-HOURS ON SITE IN THE AREAS OF CALIBRATION OF IN-PLANT INSTRUMENTATION REQUIRED BY TECHNICAL SPECIFICATIONS; CALIBRATION OF MEASURING AND TEST EQUIPMENT USED ON SAFETY-RELATED SYSTEMS; AND SURVEILLANCE OF IN-PLANT EQUIPMENT REQUIRED BY TECHNICAL SPECIFICATIONS. OF THE THREE AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN TWO AREAS; ONE APPARENT ITEM OF NONCOMPLIANCE WAS FOUND IN ONE AREA (DEFICIENCY - FAILURE TO FOLLOW SURVEILLANCE TEST PROCEDURES).

INSPECTION JANUARY 8-11 (80-02): THIS SPECIAL, UNANNOUNCED INSPECTION INVOLVED 13 INSPECTOR-HOURS ON SITE IN THE AREA OF FIRE PROTECTION/PREVENTION. OF THE AREA INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED; HOWEVER, 14 APPARENT DEVIATIONS WERE FOUND (INADEQUATE FIRE PROTECTION QA PROGRAM; SUBSTANDARD AUTOMATIC SPRINKLER SYSTEMS; STORAGE OF COMBUSTIBLES IN EAST CABLEWAY OF UNIT 1; INADEQUATE OIL CATCHMENT FACILITY FOR UNIT 2 RHR PUMPS; SIGNS TO PROHIBIT STORAGE OF COMBUSTIBLE MATERIALS NOT PROVIDED AT RIVER WATER INTAKE STRUCTURE; SUBSTANDARD FIRE DOORS BEEN CONTROL ROOM AND ADJACENT AREAS; INCORRECT TYPE FIRE DETECTORS IN CONTROL ROOM KITCHEN; INADEQUATE PROTECTION AGAINST SPRINKLER SYSTEM DISCHARGE FOR FOUR SAFETY-RELATED CABINETS - 130 FOOT ELEVATION OF CONTROL BUILDING CORRIDORS; MOBILE FIRE FIGHTING PLATFORM NOT PROVIDED FOR UNIT 1 NORTHWEST CABLE AREA; CONTROL VALVES IN DRAIN PIPING FROM REACTOR CIRCULATION M-G SETS NOT LOCKED IN CLOSED POSITION; INADEQUATE FIRE BARRIERS BETWEEN FIRE PUMP NOS. 2 AND 3; SUBSTANDARD SUPERVISION OF FIRE PROTECTION CONTROL VALVES; INCORRECT LOCATION OF AIR COMPRESSOR FOR EMERGENCY BREATHING APPARATUS; AND, SUBSTANDARD WATER SUPPLY TO SPRINKLER/STANDPIPE SYSTEM AT RIVER INTAKE STRUCTURE.

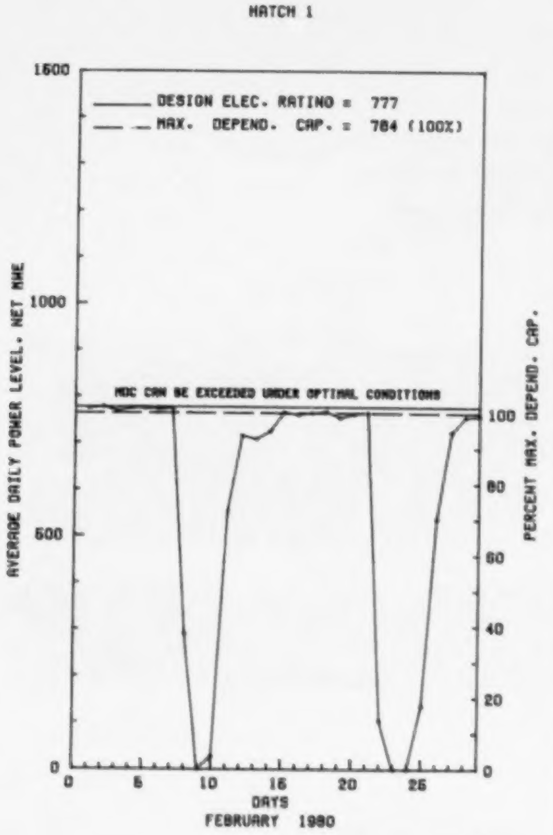
INSPECTION DECEMBER 12 - JANUARY 4 (80-04): THIS INSPECTION INVOLVED 35 INSPECTOR-HOURS ON SITE OF TECHNICAL SPECIFICATION COMPLIANCE, REPORTABLE OCCURRENCES, HOUSEKEEPING, OPERATOR PERFORMANCE, OVERALL PLANT OPERATIONS, QUALITY ASSURANCE PRACTICES, STATION AND CORPORATE MANAGEMENT PRACTICES, CORRECTIVE AND PREVENTATIVE MAINTENANCE ACTIVITIES, SITE SECURITY PROCEDURES, RADIATION CONTROL ACTIVITIES, AND SURVEILLANCE ACTIVITIES. OF THE 11 AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN TEN AREAS; ONE ITEM OF NONCOMPLIANCE WAS FOUND IN ONE AREA (INFRACTION - FAILURE TO COMPLY WITH LCO ACTION STATEMENTS).

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 4-10-80

REVIEWED BY NRR DOR: John N. Hannon **DATE:** 4-14-80

HATCH 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-321
2. Utility Contact: J. Rogen (912) 367-7781
3. Licensed Thermal Power (MWt): 2436
4. Nameplate Rating (Gross MWe): 1000 x 0.85 = 850
5. Design Electrical Rating (Net MWe): 777
6. Maximum Dependable Capacity (Gross MWe): 797
7. Maximum Dependable Capacity (Net MWe): 764
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	696.0	1,440.0	36,584.0
12. Hours Reactor Was Critical	590.4	1,220.8	27,018.8
13. Reactor Reserve Shutdown Hours	.0	.0	0
14. Hours Generator On-Line	564.7	1,145.8	25,414.2
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,266,120	2,565,280	52,898,315
17. Gross Electrical Energy (MWH)	421,080	846,010	17,116,950
18. Net Electrical Energy (MWH)	401,731	807,519	16,269,452
19. Unit Service Factor	81.1	79.6	69.6
20. Unit Availability Factor	81.1	79.6	69.6
21. Unit Capacity Factor (MDC Net)	75.5	73.4	58.3
22. Unit Capacity Factor (DER Net)	74.3	72.2	57.6
23. Unit Forced Outage Rate	18.9	20.4	16.3

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-05	02/08/80	F	57.7	H	3		IB	INSTRU	SHUTDOWN DUE TO RX SCRAM ON FALSE HIGH PRESSURE WHILE RX. PRESS. SWITCH WAS BEING VALVED IN.
80-06	02/22/80	F	67.3	B	3		CH	VALVEX	SHUTDOWN TO PERFORM DRYWELL INSPECTION AND REPAIR STEAM LEAKS.
80-07	02/25/80	F	6.3	A	1		HD	ZZZZZZ	SHUTDOWN DUE TO LOSS OF STEAM SEAL PRESSURE ON MAIN TURBINE.
80-08	02/26/80	F	.0	H	5		RD	ZZZZZZ	REDUCTION DUE TO FUEL PRECONDITIONING AND HIGH CMPF.

SUMMARY

THE UNIT WAS SHUTDOWN ON 02/08/80 DUE TO A REACTOR SCRAM ON FALSE HIGH PRESSURE WHILE THE REACTOR PRESSURE SWITCH WAS BEING VALVED IN; ON 02/22/80 TO PERFORM DRYWELL INSPECTION AND TUBE LEAK REPAIR; AND ON 02/25/80 DUE TO LOSS OF STEAM SEAL PRESSURE ON THE MAIN TURBINE. LOAD WAS REDUCED ON 02/26/80 DUE TO FUEL PRECONDITIONING AND HIGH FLUX FILL.

LEGEND

[1] Type:
F–Forced
S–Scheduled

[2] Reason:
A–Equipment Failure
B–Maintenance or Test
C–Refueling
D–Regulatory Restriction
E–Operator Training & License Examination
F–Administrative
G–Operational Error
H–Other

[3] Method:
1–Manual
2–Manual Scram
3–Automatic Scram
4–Continuations
5–Load Reduction
9–Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

HATCH 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Georgia/Appling 11 miles N of Baxley
2.	Electric Reliability Council	Member of SE Electric Reliability Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2436
5.	Design Electrical Rating (MWE Net)	784
6.	Date of Initial Criticality	July 4, 1978
7.	Date of Commercial Operation	September 5, 1979
8.	Condenser Cooling Method	Cooling Towers
9.	Condenser Cooling Water Source	Altamaha River

Utility & Contractor Information

10.	Licensee	Georgia Power Company
11.	Corporate Address	270 Peach Street, NE Atlanta, Georgia 30303
12.	Corporate Contact	J. H. Miller Executive Vice President
13.	Architect/Engineer	Southern Services, Inc./Bechtel
14.	Nuclear Steam Supply System	GE
15.	Constructor	Georgia Power Company

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. Lewis
18.	IE Senior Inspector	H. C. Dance
19.	IE Resident Inspector	R. F. Rogers (912) 367-9881
20.	Licensing Project Manager	J. N. Hannon
21.	Docket Number	50-366
22.	License Number/Date of Issue	NPF-5/June 13, 1978
23.	Public Document Room	Appling County Public Library Parker Street Baxley, Georgia 31513

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO TECHNICAL SPECIFICATION 6.8.1.C, THE INSTRUCTIONS OF HNP-2-9203 WERE NOT IMPLEMENTED BETWEEN JANUARY 1979 AND DECEMBER 1979. HOWEVER, A COMPUTER CODE WAS UTILIZED TO SATISFY THE REQUIREMENT WITHOUT CHANGING THE ASSIGNMENT SHEET TO REFLECT THIS METHOD OF OPERATION.
(8001 DEFICIENCY)

THE FIRE DOORS IN THE FIRE BARRIERS ENCLOSING THE CONTROL ROOM DO NOT FULLY MEET THE PROVISIONS OF NFPA-80 AS STATED BY THE LICENSEE'S EVALUATION OF FIRE PROTECTION PROGRAM SECTION III.C.1.2.
(8002 DEVIATION)

FLOORS ARE NOT MARKED IN THE RIVER INTAKE STRUCTURE TO PROHIBIT THE STORAGE OF COMBUSTIBLES AS STATED BY SECTION III.C.4 OF THE LICENSEE'S EVALUATION OF FIRE PROTECTION PROGRAM.
(8002 DEVIATION)

DOCUMENTATION IS NOT PROVIDED TO DEMONSTRATE THAT THE FIRE PROTECTION MODIFICATIONS WERE ACCOMPLISHED UNDER THE CONTROL OF THE QA ORGANIZATION AS STATED BY SECTION IV.B.7 OF THE LICENSEE'S EVALUATION OF FIRE PROTECTION PROGRAM.
(8002 DEVIATION)

AUTOMATIC SPRINKLER SYSTEM INSTALLATIONS DO NOT FULLY MEET THE REQUIREMENTS OF NFPA-13 AS STATED BY SECTION IV.C.3.C OF THE LICENSEE'S EVALUATION OF FIRE PROTECTION PROGRAM.
(8002 DEVIATION)

THE OIL CATCHMENT AREA BENEATH THE RHR PUMPS IS APPROXIMATELY 25% FULL OF AN OIL-WATER SOLUTION AND IS NOT FUNCTIONAL. THIS IS CONTRARY TO THE PROVISIONS OF SECTION III.C.4 OF THE LICENSEE'S EVALUATION OF FIRE PROTECTION PROGRAM.
(8002 DEVIATION)

THE UNDERGROUND SUPPLY MAIN TO THE SPRINKLER/STANDPIPE SYSTEMS WITHIN THE RIVER INTAKE STRUCTURE DOES NOT MEET THE PROVISIONS OF NFPA-24 AS STATED BY SECTION IV.C.2.A OF THE LICENSEE'S EVALUATION OF FIRE PROTECTION PROGRAM.
(8002 DEVIATION)

A NUMBER OF FIRE PROTECTION CONTROL VALVES WERE NOT PROVIDED WITH THE CORRECT TYPE OF VALVES SUPERVISION AS STATED BY SECTION IV.C.3.B OF THE LICENSEE'S EVALUATION OF FIRE PROTECTION PROGRAM.
(8002 DEVIATION)

SAFETY RELATED CABINETS AT 130 FOOT CORRIDOR OF THE CONTROL BUILDING ARE NOT PROTECTED AGAINST DISCHARGE OF WATER FROM THE SPRINKLER SYSTEM AS STATED BY SECTION IV.C.1.4.(G) OF THE EVALUATION OF FIRE PROTECTION PROGRAM.
(8002 DEVIATION)

HEAT DETECTORS ARE INSTALLED WITHIN THE CONTROL ROOM KITCHEN IN LIEU OF THE SMOKE DETECTORS STATED BY SECTION III.C.1.1 OF THE LICENSEE'S EVALUATION OF FIRE PROTECTION PROGRAM.
(8002 DEVIATION)

FIRE BARRIER HAS NOT BEEN PROVIDED BETWEEN THE NO. 2 AND NO. 3 FIRE PUMP UNITS AS STATED BY SECTION IV.C.2.C OF THE LICENSEE'S EVALUATION OF FIRE PROTECTION PROGRAM.
(8002 DEVIATION)

THE AIR COMPRESSOR FOR REFILLING THE SELF CONTAINED BREATHING APPARATUS CYLINDERS IS LOCATED IN THE WATER TREATMENT BUILDING IN LIEU OF THE DIESEL GENERATOR BUILDING AND IS NOT PROVIDED WITH AN EVAILABLE POWER SUPPLY IN THE EVENT OF LOSS OF OFF SITE POWER AS STATED BY SECTION IV.B.4.H OF THE LICENSEE'S EVALUATION OF THE FIRE PROTECTION PROGRAM.
(8002 DEVIATION)

CONTROL VALVES ON OIL DRAIN PIPING SYSTEM FROM REACTOR RECIRCULATION M-G SET ROOMS WERE NOT LOCKED CLOSED AS STATED BY TABLE 3, ITEM NO. III.43 OF THE LICENSEE'S EVALUATION OF FIRE PROTECTION PROGRAM.
(8002 DEVIATION)

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-139/ 03L-0	12/28/79	01/17/80	30 DAY LER	DIESEL GENERATOR FAILURE TO START.
80-002/ 03L-0	01/28/80	02/04/80	30 DAY LER	SINGLE MODE FAILURE DESIGN ERROR.
80-003/ 03L-0	01/17/80	01/31/80	30 DAY LER	HIGH PRESSURE COOLANT INJECTION POWER INVERTER TRIPPED.
80-004/ 03L-0	01/18/80	02/07/80	30 DAY LER	RECOMBINER INLET VALVE INOPERABLE.
80-005/ 03L-0	01/19/80	02/07/80	30 DAY LER	TECHNICAL SPECIFICATION ACTION DELAYED.
80-007/ 01T-0	01/30/80	02/05/80	14 DAY LER	DRYWELL HIGH PRESSURE SWITCH DRIFT.

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

PLANT OPERATING AT FULL POWER.

+ LAST IE SITE INSPECTION DATE: FEBRUARY 2-29, 1980

+ INSPECTION REPORT NO: 50-366/80-11

SUMMARY

INSPECTION SUMMARY

+ INSPECTION JANUARY 8-11 (80-01): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 13 INSPECTOR-HOURS ON SITE IN THE AREAS OF CALIBRATION OF IN-PLANT INSTRUMENTATION REQUIRED BY TECHNICAL SPECIFICATIONS; CALIBRATION OF MEASURING AND TEST EQUIPMENT USED ON SAFETY-RELATED SYSTEMS; AND SURVEILLANCE OF IN-PLANT EQUIPMENT REQUIRED BY TECHNICAL SPECIFICATIONS. OF THE THREE AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN TWO AREAS; ONE APPARENT ITEM OF NONCOMPLIANCE WAS FOUND IN ONE AREA (DEFICIENCY - FAILURE TO FOLLOW SURVEILLANCE TEST PROCEDURES).

INSPECTION JANUARY 8-11 (80-02): THIS SPECIAL, UNANNOUNCED INSPECTION INVOLVED 12 INSPECTOR-HOURS ON SITE IN THE AREA OF FIRE PROTECTION/PREVENTION. OF THE AREA INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED; HOWEVER, 14 APPARENT DEVIATIONS WERE FOUND (INADEQUATE FIRE PROTECTION QA PROGRAM; SUBSTANDARD AUTOMATIC SPRINKLER SYSTEMS; STORAGE OF COMBUSTIBLES IN EAST CABLEWAY OF UNIT 1; INADEQUATE OIL CATCHMENT FACILITY FOR UNIT 2 RHR PUMPS; SIGNS TO PROHIBIT STORAGE OF COMBUSTIBLE MATERIALS NOT PROVIDED AT RIVER WATER INTAKE STRUCTURE; SUBSTANDARD FIRE DOORS BEEN CONTROL ROOM AND ADJACENT AREAS; INCORRECT TYPE FIRE DETECTORS IN CONTROL ROOM KITCHEN; INADEQUATE PROTECTION AGAINST SPRINKLER SYSTEM DISCHARGE FOR FOUR SAFETY-RELATED CABINETS - 130 FOOT ELEVATION OF CONTROL BUILDING CORRIDORS; MOBILE FIRE FIGHTING PLATFORM NOT PROVIDED FOR UNIT 1 NORTHWEST CABLE AREA; CONTROL VALVES IN DRAIN PIPING FROM REACTOR CIRCULATION M-G SETS NOT LOCKED IN CLOSED POSITION; INADEQUATE FIRE BARRIERS BETWEEN FIRE PUMP NOS. 2 AND 3; SUBSTANDARD SUPERVISION OF FIRE PROTECTION CONTROL VALVES; INCORRECT LOCATION OF AIR COMPRESSOR FOR EMERGENCY BREATHING APPARATUS; AND, SUBSTANDARD WATER SUPPLY TO SPRINKLER/STANDPIPE SYSTEM AT RIVER INTAKE STRUCTURE.

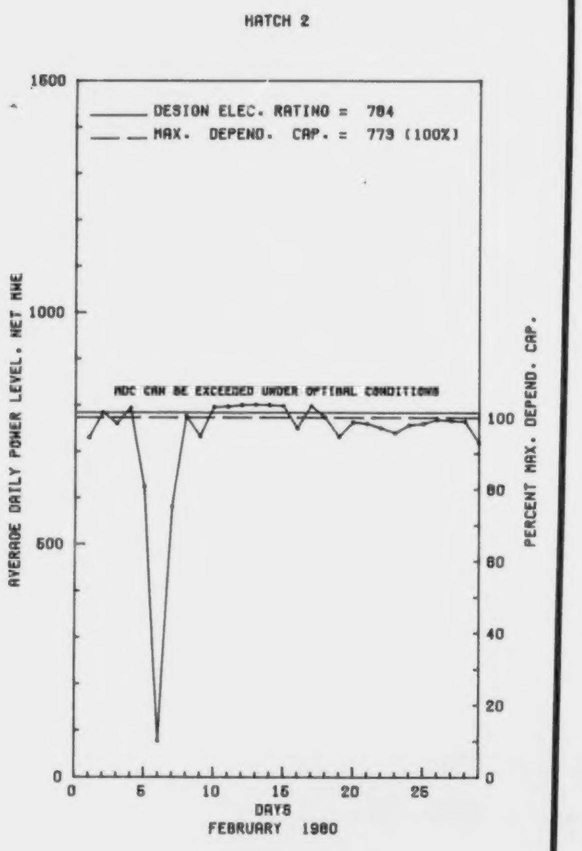
INSPECTION DECEMBER 12 - JANUARY 4 (80-04): THIS INSPECTION INVOLVED 35 INSPECTOR-HOURS ON SITE OF TECHNICAL SPECIFICATION COMPLIANCE, REPORTABLE OCCURRENCES, HOUSEKEEPING, OPERATOR PERFORMANCE, OVERALL PLANT OPERATIONS, QUALITY ASSURANCE PRACTICES, STATION AND CORPORATE MANAGEMENT PRACTICES, CORRECTIVE AND PREVENTATIVE MAINTENANCE ACTIVITIES, SITE SECURITY PROCEDURES, RADIATION CONTROL ACTIVITIES, AND SURVEILLANCE ACTIVITIES. OF THE 11 AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN TEN AREAS; ONE ITEM OF NONCOMPLIANCE WAS FOUND IN ONE AREA (INFRACTION - FAILURE TO COMPLY WITH LCO ACTION STATEMENTS).

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 4-10-80

REVIEWED BY NRR DOR: John N. Hannon **DATE:** 4-14-80

HATCH 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-366
2. Utility Contact: J. Roden (912) 367-7781
3. Licensed Thermal Power (MWt): 2436
4. Nameplate Rating (Gross MWe): 1000 × .85 = 850
5. Design Electrical Rating (Net MWe): 784
6. Maximum Dependable Capacity (Gross MWe): 806
7. Maximum Dependable Capacity (Net MWe): 773
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	696.0	1,440.0	4,273.0
12. Hours Reactor Was Critical	690.3	1,374.4	4,037.9
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	680.3	1,341.5	3,756.2
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,588,320	3,060,210	8,554,272
17. Gross Electrical Energy (MWH)	531,560	1,031,350	2,867,310
18. Net Electrical Energy (MWH)	510,082	988,878	2,746,815
19. Unit Service Factor	97.7	93.2	87.9
20. Unit Availability Factor	97.7	93.2	87.9
21. Unit Capacity Factor (MDC Net)	94.8	88.8	83.1
22. Unit Capacity Factor (DER Net)	93.5	87.6	82.0
23. Unit Forced Outage Rate	2.3	1.2	5.3

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): VENT HEADER DEFLECTOR & SURVEIL. MARCH 1 TO APRIL 20.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-04	02/05/80	F	.0	A	5		WA	PIPEXX	REDUCTION TO REPAIR PSW LEAK ON ALTEREX SYSTEM.
80-05	02/06/80	F	15.7	A	1		CH	VALVEX	SHUTDOWN DUE TO BLOWN PACKING ON SEAL WATER REGULATOR VALVE (AND LOSS OF CONDENSER VACUUM DUE TO LOSS OF SEAL WATER LOOP SEAL) OFF OF THE RFPT WATER SEAL.

SUMMARY

OPERATION WAS AT NEAR FULL POWER DURING THE REPORT PERIOD, EXCEPT FOR A SHUTDOWN ON 02/06/80 DUE TO BLOWN FACKING ON THE SEAL WATER REGULATOR VALVE. LOAD WAS REDUCED ON 02/05/80 TO REPAIR A LEAKING WATER PIPE.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

HUMBOLDT BAY

FACILITY DATA

Facility Description

1. Location (state/county) (nearest population center)	California/Humboldt 4 miles S of Eureka
2. Electric Reliability Council	WSCC
3. Type of Reactor	BWR
4. Power Level (MWT)	220
5. Design Electrical Rating (MWE Net)	65
6. Date of Initial Criticality	February 16, 1963
7. Date of Commercial Operation	August 1963
8. Condenser Cooling Method	Circulating sea water
9. Condenser Cooling Water Source	Pacific Ocean (Humboldt Bay)

Utility & Contractor Information

10. Licensee	Pacific Gas & Electric Co.
11. Corporate Address	77 Beale Street San Francisco, California 94106
12. Corporate Contact	P. A. Crane, Jr. Asst. General Counsel
13. Architect/Engineer	Bechtel
14. Nuclear Steam Supply System	GE
15. Constructor	Bechtel

Regulatory Information

16. IE Region responsible	V
17. IE Branch Chief	J. L. Crews
18. IE Senior Inspector	D. M. Sternberg
19. IE Principal Inspector	J. D. Carlson
20. Licensing Project Manager	V. L. Rooney
21. Docket Number	50-133
22. License Number/Date of Issue	DPR-7, 8/28/62
23. Public Document Room	Humboldt County Library 636 F Street Eureka, California 95501

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST (CITE RO REPORT NUMBER, IF APP.)

SYSTEMS AND COMPONENT PROBLEMS:

None.

FACILITY ITEMS (PLANS AND PROCEDURES):

The plant has been shut down since July 1976

Several items remaining to be completed during the outage have been deferred pending resolution of the seismic issues at the plant site. A date for return to operations has not been established.

MANAGERIAL ITEMS:

None

PLANT STATUS:

Plant is shutdown.

LAST IE SITE INSPECTION DATE: 12/18-20/79

INSPECTION REPORT NO: 50-133/79-04

SUMMARY

INSPECTION SUMMARY

+ No inspections conducted in February 1980.

LICENSE STATUS

Order issued 5/21/76 requiring Utility to prove seismic qualification before resuming operation of unit after outage of 7/2/76.

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
NONE				

REVIEWED BY OMPA LOEB: John Riggs DATE: 4-10-80

REVIEWED BY NRR DOR: DATE: 4/11/80

HUMBOLDT BAY

AVERAGE DAILY POWER LEVEL (MWe)

OPERATING STATUS

1. Docket Number: 50-133
2. Utility Data Prepared By: B. C. Getty (707) 443-2787
3. Licensed Thermal Power (MWt): 220
4. Nameplate Rating (Gross MWe): 77 X 0.85 = 65
5. Design Electrical Rating (Net MWe): 65
6. Maximum Dependable Capacity (Gross MWe): 0
7. Maximum Dependable Capacity (Net MWe): 0
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons
None
9. Power Level To Which Restricted, If Any (Net MWe): None
10. Reasons For Restrictions, If Any:

This ... ive

11. Hours In Reporting Period
12. Number Of Hours Reactor Was Cri
13. Reactor Reserve Shutdow
14. Hours Generato
15. Unit R
1 ... .rl)
... erated (MWH)
. Net ... gy Generated (MWH)
19. Unit S ... ce Factor
20. Unit Availability Factor
21. Unit Capacity Factor (Using MDC Net) 1/
22. Unit Capacity Factor (Using DER Net) 1/
23. Unit Forced Outage Rate
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
None
25. If Shut Down At End Of Report Period, Estimated Date of Startup: UNKNOWN

NO DECISION ON FUTURE OPERATION
SHUTDOWN INDEFINITELY

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
SEE SUMMARY BELOW									

SUMMARY

THE UNIT WAS SHUTDOWN ON 7/2/76 FOR SEISMIC MODIFICATIONS. THE UNIT REMAINS IN THE COLD SHUTDOWN CONDITION.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

INDIAN POINT 1

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	New York/Westchester 25 miles N of New York City
2.	Electric Reliability Council	NPCC
3.	Type of Reactor	PWR/fossil superheat
4.	Power Level (MWT)	615
5.	Design Electrical Rating (MWE Net)	265
6.	Date of Initial Criticality	August 2, 1962
7.	Date of Commercial Operation	October 1962
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Hudson River

Utility & Contractor Information

10.	Licensee	Consolidated Edison Company of New York, Inc.
11.	Corporate Address	4 Irving Place New York, New York
12.	Corporate Contact	W. Cahill, Vice President
13.	Architect/Engineer	Utility
14.	Nuclear Steam Supply System	B&W
15.	Constructor	Utility

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Senior Inspector	R. Keimig
19.	IE Resident Inspector	T. Rebelowski (914-739-9360) FTS 656-9220
20.	Licensing Project Manager	L. Olshan x27144
21.	Docket Number	50-03
22.	License Number/Date of Issue	DPR-5, 3/26/62
23.	Public Document Room	White Plains Public Library 100 Martine Avenue White Plains, New York 10601

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

None

FACILITY ITEMS (PLANS AND PROCEDURES):

Order to Show Cause why the operating provision of License No. DPR-5 should not be revoked and why the licensee should not submit a plan to decommission the facility was issued on February 11, 1980.

MANAGERIAL ITEMS:

None

PLANT STATUS.

Defueled.

LAST IE SITE INSPECTION DATE: 7/24-27/79

INSPECTION REPORT NO: 50-03/79-02

SUMMARY

INSPECTION SUMMARY

None.

LICENSE STATUS

The unit produced electricity to the expiration date (October 31, 1974) of delay granted by the AEC for installation of an ECCS system. The ECCS system was not subsequently installed and the unit was defueled on January 30, 1976.

On February 11, 1980, the NRC requested the utility to show cause as to why the operating authority of the license should not be revoked and why the licensee should not submit a plan within 120 days of this order to decommission the unit.

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
NONE				

REVIEWED BY OMPA LOEB: T.C. Cwtula DATE: 4·10·80

REVIEWED BY NRR DOR: DATE: 4-11-80

INDIAN POINT 1

AVERAGE DAILY POWER LEVEL (MWe)

OPERATING STATUS

1. Docket Number: 50-003
2. Utility Data Prepared By: L. J. Kawula (914)-694-6000 x209
3. Licensed Thermal Power (MWt): 615
4. Nameplate Rating (Gross MWe): 275
5. Design Electrical Rating (Net MWe): 265
6. Maximum Dependable Capacity (Gross MWe): 0
7. Maximum Dependable Capacity (Net MWe): 0
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons
None
9. Power Level To Which Restricted, If Any (Net MWe): None
10. Reasons For Restrictions, If Any:

	This Month	Yr.-to-Date
11. Hours In Reporting Period		
12. Number Of Hours Reactor Was Critical		
13. Reactor Reserve Shutdown Hours		
14. Hours Generator On-Line		
15. Unit Reserve Shutdown Ho		
16. Gross Thermal E		
17. Gros		
20. Unit A		
21. Unit Capacity Factor (U Net) 1/		
22. Unit Capacity Factor (Using DER Net) 1/		
23. Unit Forced Outage Rate		

NO DECISION ON FUTURE OPERATION
SHUTDOWN INDEFINITELY
ALL FUEL UNLOADED.

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
None
25. If Shut Down At End Of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
SEE SUMMARY BELOW									

SUMMARY

UNIT SHUTDOWN ON OCTOBER 31, 1974. ALL FUEL HAS BEEN UNLOADED; NO DECISION ON FUTURE OPERATION HAS BEEN MADE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

INDIAN POINT 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	New York/Westchester 25 miles N of New York City
2.	Electric Reliability Council	NPCC
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2758
5.	Design Electrical Rating (MWE Net)	873
6.	Date of Initial Criticality	May 22, 1973
7.	Date of Commercial Operation	August 1973
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Hudson River

Utility & Contractor Information

10.	Licensee	Consolidated Edison Company of New York, Inc.
11.	Corporate Address	4 Irving Place New York, New York
12.	Corporate Contact	W. J. Cahill, Jr., Vice President
13.	Architect/Engineer	UE&C
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	J. A. Jones/UE&C

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Senior Inspector	R. Keimig
19.	IE Resident Inspector	T. Rebelowski (914-739-9360) FTS 656-9220
20.	Licensing Project Manager	L. Olshan (X27144)
21.	Docket Number	50-247
22.	License Number/Date of Issue	DPR-26, 10/19/71
23.	Public Document Room	White Plains Public Library 100 Martine Avenue White Plains, New York 10601

INSPECTION STATUS

ENFORCEMENT STATUS

THE LICENSEE ALLOWED THE EFFECTIVENESS AND INTEGRITY OF THE UNIT 2 SCREENWELL BARRIER TO BE COMPROMISED IN THAT THE CONTINUAL AREA SURVEILLANCE PATROL WAS NOT POSTED BETWEEN 3:00 P.M. AND 6:00 P.M., APRIL 15, 1979 TO MAINTAIN A HIGH LEVEL OF ASSURANCE AGAINST UNDETECTED PENETRATION OF THE PROTECTED AREA PERIMETER.
(7922 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

None

FACILITY ITEMS (PLANS AND PROCEDURES):

Next scheduled refueling outage 1/1/81.

MANAGERIAL ITEMS:

(1) Confirmatory order issued February 11, 1980 concerning implementation of TMI Lessons Learned and Emergency Preparedness.

(2) The licensee has appointed a new Instrumentation and Control Supervisor and a new Security Supervisor.

PLANT STATUS:

Unit 2 returned to service on 2/10 after completion of turbine inspection and repairs.

LAST IE SITE INSPECTION DATE: 2/2/80

INSPECTION REPORT NO: 50-247/80-01

SUMMARY

INSPECTION SUMMARY

50-247/79-21 - Oct 14-Nov 10: Routine inspection (41 hours by 2 resident inspectors) of plant operations including: shift logs and records; tours of the facility; observations of physical security; Emergency Planning operations; surveillance testing; review of licensee events; and, follow-up on previous inspection items. No items of noncompliance were identified.

50-247/79-23 - Nov 11-Dec 8: Routine inspection (25 hours by 2 resident inspectors) of plant operations including: shift logs and records; tours of the facility; observations of physical security; Emergency Planning Drill; Reactor Coolant System Leakage and Pressurizer Safety Valve testing; periodic reports; and changes in station management staff. One item of noncompliance was identified (Failure to recognize a Quadrant Power Tilt calculation result in excess of Technical Specification limits).

50-247/79-24 - Nov 15-16: Routine, unannounced inspection of the licensee's response to IE Bulletin 79-19 including maintenance of regulatory requirements, training, audits and procedures. The inspection involved 28 hours onsite by two NRC region-based inspectors. Of the five areas inspected, one item of noncompliance was found (Failure to have copies of all documents referenced by the Certificate of Compliance).

REPORTS RECEIVED FROM LICENSEE

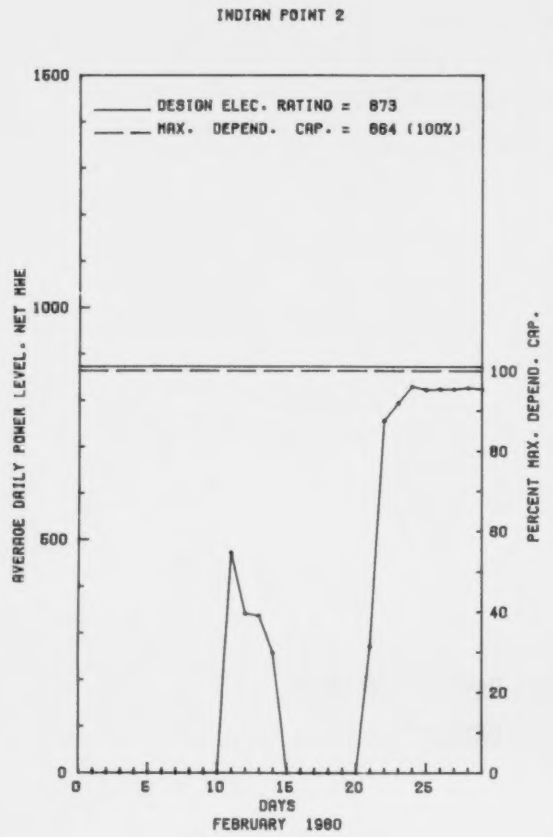
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-023/03L	11/09/79	12/07/79	30 Day LER	Containment isolation valve PCV-1217 would not shut
79-024/03L	12/02/79	12/28/79	30 Day LER	Letdown relief valve no. 203 found leaking

REVIEWED BY OMPA LOEB: T. C. Cintula **DATE:** 4•10•80

REVIEWED BY NRR DOR: **DATE:** 4-11-80

INDIAN POINT 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-247
2. Utility Contact: E. EICH (914) 694-6000 2 I.P.
3. Licensed Thermal Power (MWt): 2758
4. Nameplate Rating (Gross MWe): 1126 x 0.9 = 1013
5. Design Electrical Rating (Net MWe): 873
6. Maximum Dependable Capacity (Gross MWe): 900
7. Maximum Dependable Capacity (Net MWe): 864
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): 834
10. Reasons for Restrictions, If Any:
REMOVAL OF NO. 3 DISC ON NO. 23 LOW PRES. TURBINE ROTOR.

ITEM 21 CALCULATED BY USING A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	696.0	1,440.0	49,681.0
12. Hours Reactor Was Critical	347.2	609.3	32,903.8
13. Reactor Reserve Shutdown Hours	339.3	821.1	1,062.5
14. Hours Generator On-Line	297.1	559.3	31,948.5
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	693,616	1,354,615	82,187,141
17. Gross Electrical Energy (MWH)	206,500	390,320	25,523,406
18. Net Electrical Energy (MWH)	190,752	362,257	24,327,367
19. Unit Service Factor	42.7	38.8	64.3
20. Unit Availability Factor	42.7	38.8	64.3
*21. Unit Capacity Factor (MDC Net)	31.7	29.1	56.8
22. Unit Capacity Factor (DER Net)	31.4	28.4	56.1
23. Unit Forced Outage Rate	35.1	22.4	8.4

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each).
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
1	01/11/80	S	237.9	A	4		HA	TURBIN	UNIT SHUTDOWN FOR L.P. TURBINE INSPECTION.
2	02/14/80	F	134.4	A	2		HC	HTEXCH	CONDENSER TUBE LEAKS.
3	02/20/80	F	4.0	A	3		HJ	HTEXCH	24 S/G LOW LEVEL.
4	02/20/80	F	7.1	A	3		HJ	HTEXCH	21 S/G LOW LEVEL.
5	02/20/80	F	15.5	A	3		CH	VALVEX	24 S/G HIGH LEVEL-REPAIRS MADE TO NO. 24 FEEDWATER REGULATOR.

SUMMARY

THE UNIT RETURNED TO POWER ON 02/10/80 FROM A TURBINE INSPECTION AND REPAIR OUTAGE, AND OPERATED AT REDUCED LOADS BECAUSE OF HIGHER THAN NORMAL CHLORIDES IN THE CONDENSATE. THE UNIT WAS AGAIN SHUTDOWN ON 02/14/80 DUE TO A SUDDEN INCREASE IN CHLORIDES FROM ALL CONDENSER SEXTANTS. FOLLOWING REPAIR OF A DOUBLE-ENDED TUBE BREAK IN THE NO. 24 CONDENSER, AND SEVERAL LESSER LEAKS IN OTHER CONDENSERS, THE UNIT WAS RETURNED TO SERVICE ON 02/20/80. STEAM GENERATOR LEVEL TRIPS OCCURRED ON STARTUP.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

INDIAN POINT 3

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	New York/Westchester 25 miles N of New York City
2.	Electric Reliability Council	NPCC
3.	Type of Reactor	PWR
4.	Power Level (MWT)	3025
5.	Design Electrical Rating (MWE Net)	965
6.	Date of Initial Criticality	April 6, 1976
7.	Date of Commercial Operation	August 30, 1976
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Hudson River

Utility & Contractor Information

10.	Licensee	Power Authority of the State of New York, Inc.
11.	Corporate Address	10 Columbus Circle New York, New York
12.	Corporate Contact	G.T. Barry General Manager
13.	Architect/Engineer	UE&C
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	WEDCO

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Senior Inspector	R. Keimig
19.	IE Resident Inspector	T. Rebelowski (914-739-9360) FTS 656-9220
20.	Licensing Project Manager	L. Olshan x27144
21.	Docket Number	50-286
22.	License Number/Date of Issue	DPR-64, 12/12/75
23.	Public Document Room	White Plains Public Library 100 Martine Avenue White Plains, New York 10601

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

None

FACILITY ITEMS (PLANS AND PROCEDURES):

None

MANAGERIAL ITEMS:

Confirmatory Order issued February 11, 1980 concerning implementation of TMI Lessons Learned and Emergency Preparedness.

PLANT STATUS:

Unit 3 has completed refueling and is undergoing low power testing.

LAST IE SITE INSPECTION DATE: 1/19/80

INSPECTION REPORT NO: 50-286/80-01

SUMMARY

INSPECTION SUMMARY

Summaries of the following reports are not yet available at this time and will be provided subsequently.

September 30-October 27, 1979 - 50-286/79-23
October 23-25, 1979 - 50-286/79-25
November 13, 25, 1979 - 50-286/79-26
October 28-November 25, 1979 - 50-286/79-27
November 27, 1979 - 50-286/79-28
November 25-December 22, 1979 - 50-286/79-29
December 10-14, 1979 - 50-286/79-30
December 23, 1979 - 50-286/79-31

REPORTS RECEIVED FROM LICENSEE

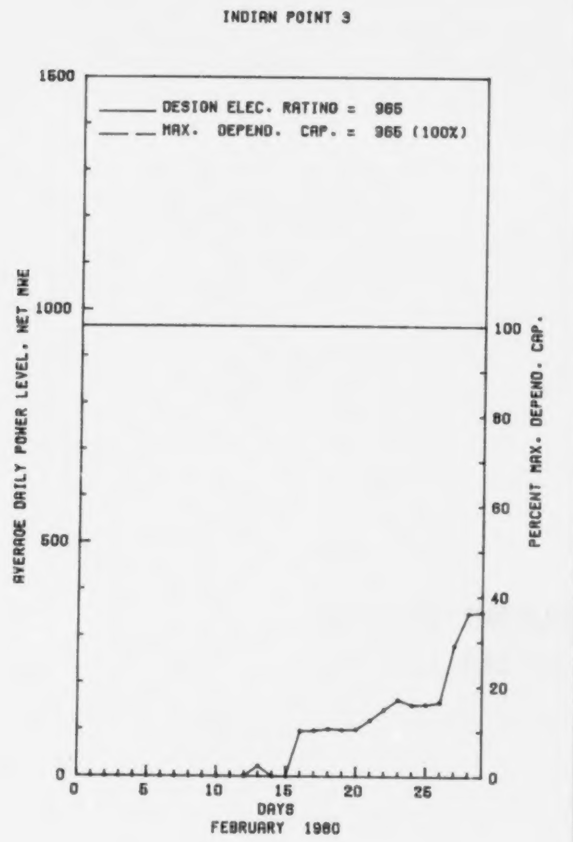
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-013/ 01T	11/27/79	11/28/79	14 Day LER	Refueling surveillance test found pressurizer safety valve out of spec
79-014/ 03L	12/02/79	12/28/79	30 Day LER	Hydrogen recombiner surveillance test indicated high dilute temperature (3PT-R12)
79-015/ 01T	11/27/79	11/28/79	14 Day LER	Refueling surveillance test found accumulator level transmitters out of calibration
79-016/ 01T	11/27/79	12/07/79	14 Day LER	Westinghouse notified licensee of possible nonconservative feature in Appendix K LOCA/ECCS analysis related to fuel burst
79-017/ 03L	12/10/79	01/09/80	30 Day LER	Refueling water storage TK calibration test found low level limit alarm out of specification
79-018/ 03L	11/23/79	12/21/79	30 Day LER	Defective snubbers have been identified during surveillance testing (3PT-R38)

REVIEWED BY OMPA LOEB: T.C. Cintula DATE: 4·10·80

REVIEWED BY NRR DOR: DATE: 4-11-80

INDIAN POINT 3

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-286
2. Utility Contact: C. Connell (914) 739-8200 x242
3. Licensed Thermal Power (MWt): 3025
4. Nameplate Rating (Gross MWe): 1126 x 0.9 = 1013
5. Design Electrical Rating (Net MWe): 965
6. Maximum Dependable Capacity (Gross MWe): 1000
7. Maximum Dependable Capacity (Net MWe): 965
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): 917
10. Reasons for Restrictions, If Any: TWO BLADING ROWS REMOVED ON L.P. TURBINE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	696.0	1,440.0	30,697.0
12. Hours Reactor Was Critical	479.2	529.9	22,114.8
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	347.3	347.3	21,380.1
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	298,849	298,849	56,403,316
17. Gross Electrical Energy (MWH)	67,340	67,340	18,432,441
18. Net Electrical Energy (MWH)	57,335	57,335	17,700,772
19. Unit Service Factor	49.9	24.1	69.6
20. Unit Availability Factor	49.9	24.1	69.6
21. Unit Capacity Factor (MDC Net)	9.0	4.3	62.9
22. Unit Capacity Factor (DER Net)	8.5	4.1	59.8
23. Unit Forced Outage Rate	8.6	8.6	4.3

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
36	09/14/79	S	261.7	C	4				REFUELING OUTAGE.
1	02/11/80	F	32.5	A	3		HB	HTEXCH	STEAM LEAK ON MSR.
2	02/13/80	S	54.5	B	1		HA	TURBIN	MAIN TURBINE TRIP SURVEILLANCE.

SUMMARY

THE UNIT WAS BROUGHT ON-LINE ON 02/11/80 FROM A REFUELING OUTAGE, BUT WAS SHUTDOWN AGAIN BECAUSE OF STEAM LEAKS OUT THE VENTS OF THE MOISTURE SEPARATOR RELOADERS. THE UNIT WAS RETURNED TO SERVICE ON 02/13/80 FOR ABOUT EIGHT HOURS AND WAS THEN TAKEN OFF-LINE FOR TURBINE OVERSPEED TESTING. POWER ESCALATION WAS ROUTINE FROM 02/16/80 THROUGH THE END OF THE REPORT PERIOD.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

KEWAUNEE

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Wisconsin/Kewaunee 27 miles E of Green Bay
2.	Electric Reliability Council	N/A
3.	Type of Reactor	PWR
4.	Power Level (MWT)	1650
5.	Design Electrical Rating (MWE Net)	535
6.	Date of Initial Criticality	March 7, 1974
7.	Date of Commercial Operation	June 1974
8.	Condenser Cooling Method	Once through condenser and return to Lake Michigan
9.	Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10.	Licensee	Wisconsin Public Service Corp.
11.	Corporate Address	P. O. Box 1200 Green Bay, Wisconsin 54305
12.	Corporate Contact	E. R. Mathews Power Supply & Eng.
13.	Architect/Engineer	Fluor Power Services Corp.
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Pioneer Services & Eng. Co.

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Senior Inspector	R. F. Warnick
19.	IE Principal Inspector	N. C. Choules
20.	Licensing Project Manager	R. B. Licciardo X28362
21.	Docket Number	50-305
22.	License Number/Date of Issue	DPR-43 issued 12/21/73
23.	Public Document Room	Kewaunee Public Library 822 Juneau Street Kewaunee, Wisconsin 54216

INSPECTION STATUS

ENFORCEMENT STATUS

ENVIRONMENTAL TECHNICAL SPECIFICATION 2.2.2 REQUIRES THAT THE AVERAGE INCREMENTAL INCREASE IN THE CONCENTRATION OF TOTAL SOLIDS IN THE CIRCULATING WATER RESULTING FROM THE NEUTRALIZING TANK DISCHARGE SHALL NOT EXCEED 2.0 MG/L. CONTRARY TO THE ABOVE REQUIREMENT, ON JUNE 7, 1979, A NEUTRALIZING TANK BATCH WITH A CALCULATED INCREMENTAL CONCENTRATION OF 2.429 MG/L OF NONRADIOACTIVE TOTAL SOLIDS IN THE CIRCULATING WATER SYSTEM WAS RELEASED TO THE PRE-TREATMENT LAGOON/CIRCULATING WATER SYSTEM.
(7920 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

+ The plant was operating at 100% power at the end of the month.

+LAST IE SITE INSPECTION DATE: January 8-11, 1980

+ INSPECTION REPORT NO: 80-01

SUMMARY

INSPECTION SUMMARY

+ Inspection on December 3-7, (79-20): Routine, unannounced inspection of emergency planning, including: licensee internal audits; coordination with offsite support agencies; emergency facilities, supplies, and equipment; medical arrangement; training; emergency plan and implementing procedures; tests and drills; actions on previous environmental events; confirmatory measurements, including reviews of results of previous sample collection; quality control of analytical measurements; and collection of effluent samples for future comparative analyses. The inspection involved 33 inspector hours onsite by one NRC inspector. Of the 11 areas inspected, no apparent items of noncompliance or deviations were identified in 10 areas; one apparent item of noncompliance (release of total solids in excess of Technical Specification limit of 2.0 mg/l) was identified in one area.

+ Inspection on January 8-11, (80-01): Routine, announced inspection of an unresolved item, plant operations, nonroutine events, and IE Bulletin and Circular followup. The inspection involved 26 inspector hours onsite by one NRC inspector. The inspector was accompanied by one NRC inspector in training. No items of noncompliance or deviations were identified.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

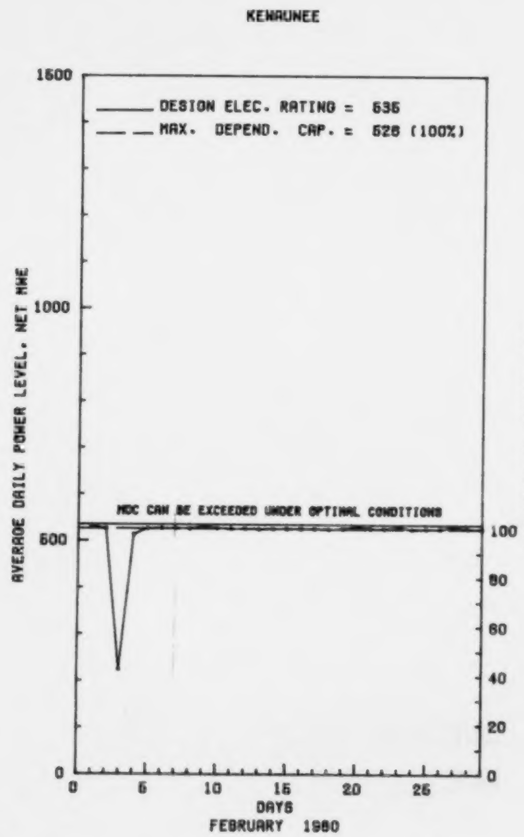
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
80-01/ 03L-0	01/03/80	01/30/80	30 Day LER	ASV door found blocked open.
80-02/ 03L-0	01/17/80	02/15/80	30 Day LER	A fault occurred in the reserve auxiliary transformer.
80-03/ 03L-0	01/17/80	02/15/80	30 Day LER	CWST inventory decreased to less than TS limit.
80-04/ 03L-0	01/17/80	02/15/80	30 Day LER	DG in a low lube condition with oil 1/2 inch low.

REVIEWED BY OMPA LOEB: T.C. Cintula DATE: 4·10·80

REVIEWED BY NRR DOR: [illegible] DATE: [illegible]

KEWAUNEE

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-305
2. Utility Contact: J. Wallace (414) 388-2560 x207
3. Licensed Thermal Power (MWt): 1650
4. Nameplate Rating (Gross MWe): 622 x 0.9 = 560
5. Design Electrical Rating (Net MWe): 535
6. Maximum Dependable Capacity (Gross MWe): 565
7. Maximum Dependable Capacity (Net MWe): 526
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

ITEM 21 CALCULATED BY USING A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	696.0	1,440.0	58,041.0
12. Hours Reactor Was Critical	693.8	1,173.0	42,175.5
13. Reactor Reserve Shutdown Hours	.0	152.7	2,330.5
14. Hours Generator On-Line	691.9	1,155.8	41,151.8
15. Unit Reserve Shutdown Hours	.0	.0	10.0
16. Gross Thermal Energy (MWH)	1,124,815	1,837,968	62,989,071
17. Gross Electrical Energy (MWH)	375,600	612,500	20,796,300
18. Net Electrical Energy (MWH)	359,111	584,991	19,784,069
19. Unit Service Factor	99.4	80.3	82.2
20. Unit Availability Factor	99.4	80.3	82.3
21. Unit Capacity Factor (MDC Net)	98.1	77.2	76.4
22. Unit Capacity Factor (DER Net)	96.4	75.9	73.9
23. Unit Forced Outage Rate	.6	19.7	5.8

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
REFUELING: MAY 10, 1980: SIX WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
3	02/03/80	F	4.1	A	3		HH	VALVOP	SOLENOID VALVE FAILURE ON 1B MAIN FEEDWATER CONTROL VALVE AIR SYSTEM CAUSED THE VALVE TO SHUT RESULTING IN A LOW STEAM GENERATOR WATER LEVEL REACTOR TRIP.

SUMMARY

OPERATION WAS AT NEAR FULL POWER DURING THE REPORT PERIOD, EXCEPT FOR A REACTOR TRIP ON LOW STEAM GENERATOR WATER LEVEL ON 02/03/80.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram.
- 3--Automatic Scram.
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

LACROSSE

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Wisconsin/Vernon 19 miles S of LaCrosse
2.	Electric Reliability Council	MARCA
3.	Type of Reactor	BWR
4.	Power Level (MWT)	165
5.	Design Electrical Rating (MWE Net)	50
6.	Date of Initial Criticality	July 11, 1967
7.	Date of Commercial Operation	November 1, 1969
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Mississippi River

Utility & Contractor Information

10.	Licensee	Dairyland Power Cooperative
11.	Corporate Address	2615 East Avenue South La Crosse, Wisconsin
12.	Corporate Contact	F. Linder
13.	Architect/Engineer	Sargent & Lundy
14.	Nuclear Steam Supply System	Allis-Chalmers
15.	Constructor	Maxson Construction

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	D. C. Boyd
18.	IE Senior Inspector	R. F. Warnick
19.	IE Principal Inspector	K. Ridgway
20.	Licensing Project Manager	J. J. Shea
21.	Docket Number	50-409
22.	License Number/Date of Issue	DPR-45 issued 8/28/73
23.	Public Document Room	LaCrosse Public Library 800 Main Street LaCrosse, Wisconsin 54601

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

Plant is operating routinely.

LAST IE SITE INSPECTION DATE: December 3-6, 1979

INSPECTION REPORT NO: 79-22

INSPECTION SUMMARY

+ No inspections conducted.

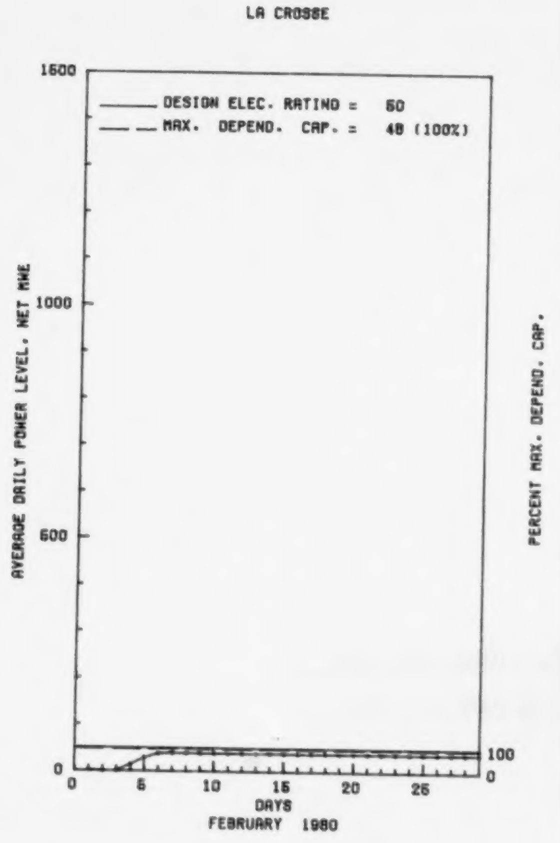
REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
NONE				

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 4-10-80

REVIEWED BY NRR DOR: **DATE:**

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-409
2. Utility Contact: L. S. Goodman (608) 689-2331
3. Licensed Thermal Power (MWt): 165
4. Nameplate Rating (Gross MWe): 76.8 x 0.85 =65
5. Design Electrical Rating (Net MWe): 50
6. Maximum Dependable Capacity (Gross MWe): 50
7. Maximum Dependable Capacity (Net MWe): 48
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	696.0	1,440.0	90,555.0
12. Hours Reactor Was Critical	638.1	1,382.1	59,188.2
13. Reactor Reserve Shutdown Hours	.0	.0	478.0
14. Hours Generator On-Line	621.5	1,365.5	54,378.5
15. Unit Reserve Shutdown Hours	.0	.0	79.0
16. Gross Thermal Energy (MWH)	84,574	188,577	7,473,356
17. Gross Electrical Energy (MWH)	25,038	55,948	2,255,072
18. Net Electrical Energy (MWH)	23,505	52,647	2,085,382
19. Unit Service Factor	89.3	94.8	60.0
20. Unit Availability Factor	89.3	94.8	60.1
21. Unit Capacity Factor (MDC Net)	70.4	76.2	48.0
22. Unit Capacity Factor (DER Net)	67.5	73.1	46.1
23. Unit Forced Outage Rate	10.7	5.2	6.4

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): EQUIPMENT INSTALLATION PER NRC - 03/10/80.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-01	02/01/80	F	74.5	A	3		EB	ELECON	MOMENTARY LOW VOLTAGE SIGNAL AT TURBINE BUILDING 480V 1A MOTOR CONTROL CENTER CAUSED SCRAM. A THOROUGH CHECK OF PLANT ELECTRICAL SYSTEM WAS CONDUCTED.

SUMMARY

OPERATION WAS ROUTINE DURING THE REPORT PERIOD, AT 85% RATED THERMAL POWER. THE 02/80 REFUELING OUTAGE HAS BEEN POSTPONED PENDING RECEIPT OF EQUIPMENT TO BE INSTALLED PER NUREG-0578. ON 02/01/80, THE REACTOR SCRAMMED ON A SIGNAL INDICATING LOW VOLTAGE AT THE TURBINE BUILDING 480V MOTOR CONTROL CENTER 1A. ALTHOUGH THE CAUSE OF THE UNDERVOLTAGE IS UNKNOWN, VIBRATION MAY HAVE CONTRIBUTED TO CONTACT ACTION. THE UNIT WAS RETURNED TO SERVICE ON 02/03/80.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examinatior
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

MAINE YANKEE

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Maine/Lincoln 10 miles N of Bath (near Wiscasset)
2.	Electric Reliability Council	NPCC
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2630
5.	Design Electrical Rating (MWE Net)	825
6.	Date of Initial Criticality	October 23, 1972
7.	Date of Commercial Operation	December 28, 1972
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Back River (Atlantic)

Utility & Contractor Information

10.	Licensee	Maine Yankee Atomic Power Co.
11.	Corporate Address	20 Turnpike Road Westboro, Massachusetts 01581
12.	Corporate Contact	D. W. Edwards, Assistant to Vice President
13.	Architect/Engineer	Stone & Webster
14.	Nuclear Steam Supply System	Combustion Engineering
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	IE Senior Inspector	T. T. Martin
19.	IE Principal Inspector	W. J. Lazarus
20.	Licensing Project Manager	M. B. Fairtile
21.	Docket Number	50-309
22.	License Number/Date of Issue	DPR-36, 9/15/72
23.	Public Document Room	Wiscasset Public Library High Street Wiscasset, Maine 04578

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS:

- The reactor was made critical on March 9 after a 7-week refueling outage.
- LAST IE SITE INSPECTION DATE: 3/7/80
- INSPECTION REPORT NO: 50-309/80-04

SUMMARY

INSPECTION SUMMARY

- 50-309/79-19 - Nov 29-30: Special, unannounced inspection by a region-based inspector (16 hr. onsite) of licensee's response to IE Bulletin No. 79-19 including a review of copies of DOT and NRC regulations maintained onsite, procedures, training of personnel, records, and audits relating to the transfer, packaging, and shipping of low-level radioactive waste material. The inspector also reviewed licensee radiation protection training, surveys, and preparations for a scheduled refueling outage. No items of noncompliance were identified.
- 50-309/80-01 - Jan 21-25: Routine, unannounced inspection of refueling preparation; refueling activities; plant operations; licensee performance of review and audits and followup on selected Licensee Event Reports (LERs). The inspection involved 38 hours on site by two region-based inspectors. Two items of noncompliance were identified (failure to comply with Radiation Work Permit requirement; and, audible count rate monitor not operating in refueling cavity area).
- 50-309/80-02 - Jan 21-25: Routine, unannounced inspection by a region-based inspector (36 hr. onsite) of radiation protection during refueling including licensee action on IE circulars, procedures, advanced planning and preparation, training, exposure control, respiratory protection, posting and control, facility tours, and surveys. Upon arrival, a tour of the radiation control area was performed to observe radiation safety practices. One item of noncompliance was identified (failure to establish procedures).

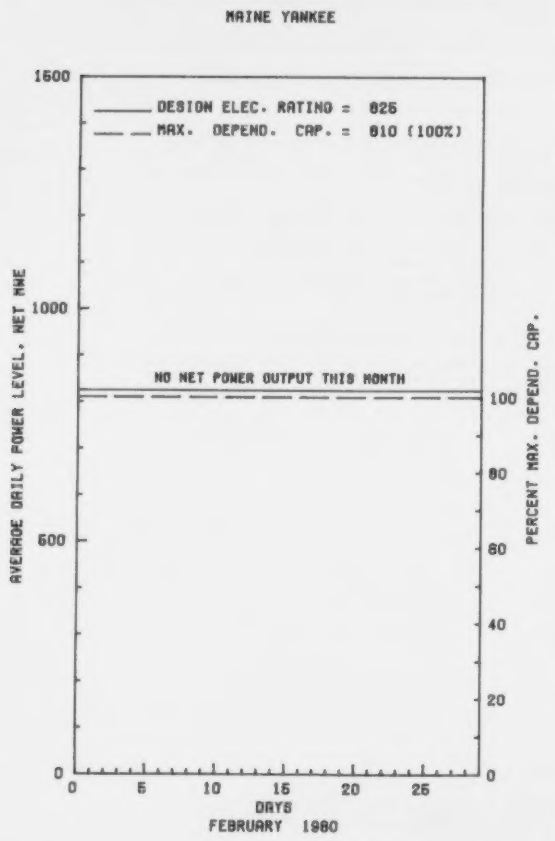
REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
80-002/ 03L	01/08/80	01/31/80	30 Day LER	Containment personnel hatch o-ring seal found leaking during surveillance test
80-003/ 03L	01/15/80	02/04/80	30 Day LER	Diesel generator output breaker failure
80-004/ 01T	01/31/80	02/14/80	14 Day LER	Fuel pin cladding failure

REVIEWED BY OMPA LOEB: MR Beebe **DATE:** 04/10/80

REVIEWED BY NRR DOR: Morton B. Fairtile **DATE:** 4/11/80

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-309
2. Utility Contact: Robert H. Groce (617) 366-9011
3. Licensed Thermal Power (MWt): 2630
4. Nameplate Rating (Gross MWe): 900 x 0.9 = 810
5. Design Electrical Rating (Net MWe): 825
6. Maximum Dependable Capacity (Gross MWe): 850
7. Maximum Dependable Capacity (Net MWe): 810

CAPACITY FACTORS CALCULATED WITH A WEIGHTED AVERAGE

8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	696.0	1,440.0	64,066.6
12. Hours Reactor Was Critical	.0	263.4	51,473.3
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	.0	263.1	49,780.3
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	442,499	105,674,582
17. Gross Electrical Energy (MWH)	0	139,660	34,712,840
18. Net Electrical Energy (MWH)	0	131,040	32,958,226
19. Unit Service Factor	.0	18.3	77.7
20. Unit Availability Factor	.0	18.3	77.7
21. Unit Capacity Factor (MDC Net)	.0	11.2	66.9
22. Unit Capacity Factor (DER Net)	.0	11.0	64.6
23. Unit Forced Outage Rate	.0	.0	6.9

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 03/01/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
8-79-4	01/11/80	S	696.0	C	4				CYCLE 4/5 REFUELING SHUTDOWN CONTINUES.

SUMMARY

THE UNIT REMAINED SHUTDOWN THROUGHOUT THE REPORT PERIOD FOR REFUELING.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram.
- 3--Automatic Scram.
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

MILLSTONE 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Connecticut/New London 5 miles SW of New London
2.	Electric Reliability Council	NPCC
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2011
5.	Design Electrical Rating (MWE Net)	660
6.	Date of Initial Criticality	October 26, 1970
7.	Date of Commercial Operation	March 1971
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Long Island Sound

Utility & Contractor Information

10.	Licensee	Northeast Nuclear Energy Company
11.	Corporate Address	P. O. Box 270 Hartford, Connecticut
12.	Corporate Contact	W. G. Counsil, Vice President
13.	Architect/Engineer	Ebasco
14.	Nuclear Steam Supply System	GE
15.	Constructor	Ebasco

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	IE Senior Inspector	E. McCabe (FTS: 244-2000)
19.	IE Resident Inspector	J. Shedlosky (203) 447-3179
20.	Licensing Project Manager	J. J. Shea
21.	Docket Number	50-245
22.	License Number/Date of Issue	DPR-21, 10/7/70
23.	Public Document Room	Waterford Public Library Rope Ferry Road Route 156 Waterford, Connecticut 06385

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

* The isolation condenser was returned to standby service on February 11, following modifications to the steam line containment penetration pipe restraint. During the period that the system was isolated the reactor operated at a maximum power level of 40%. On February 21, the "A" Core Spray was considered inoperable when several damaged pipe restraints were identified.

PLANT STATUS:

* The plant has operated at 40% with the isolation condenser system out of service through February 11. Representative offgas and stack release rates at 100% power are 135,000 microcuries per second and 320 microcuries per second, respectively.

* LAST IE SITE INSPECTION DATE: 2/29/80

* INSPECTION REPORT NO: 50-245/80-03

SUMMARY

INSPECTION SUMMARY

* 50-245/79-29 - Dec 1-31: Routine, onsite regular and backshift inspection by two resident inspectors (60.5 hours). Areas inspected included the control rooms and the accessible portions of the primary containment, reactor, turbine, radioactive waste, gas turbine generator, and intake buildings; radiation protection; physical security; fire protection; plant operating records; surveillance testing; calibration; maintenance; core power distribution limits; and NRC reporting requirements. Noncompliances: None.

REPORTS RECEIVED FROM LICENSEE

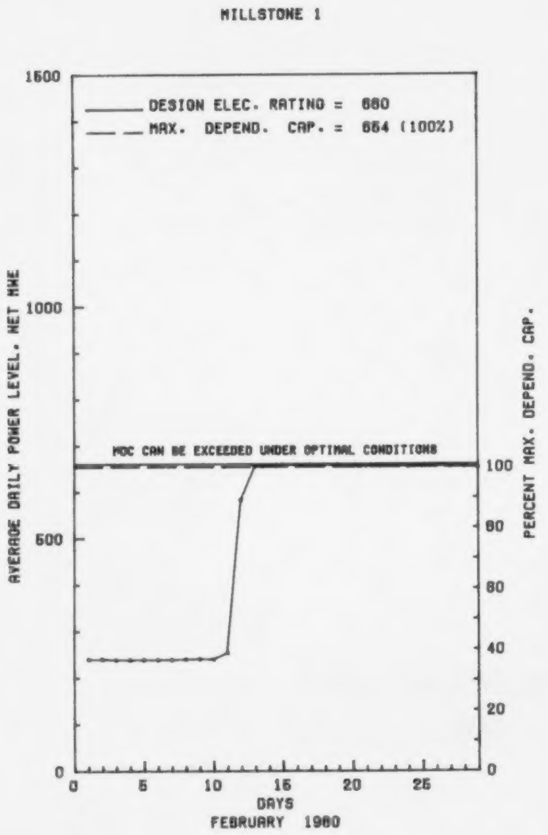
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-002E	11/06/79	11/16/79	Environmental Deviation	Floor drain sample tank curie content found greater than TS limit. Tank was flushed refilled to within TS limit.
79-031/ 03L	10/16/79	11/15/79	30 Day LER	FWCI sybsystem inoperable to install two seismic restraints
79-032/ 03L	11/06/79	12/03/79	30 Day LER	Two of four high drywell pressure switch setpoints found slightly above TS limits
79-033/ 03L	11/13/79	12/04/79	30 Day LER	One of sixteen main steam line high flow switches found set higher than TS setpoint
79-034/ 03L	11/15/79	12/14/79	30 Day LER	Failure of LPCI inboard isolation valve which resulted in the loss of ability to inject LPCI to B recirculation loop
79-042/ 03L	12/28/79	01/25/80	30 Day LER	Safety injection tank No. 1 level decreased below TS required level of 55 percent to 54.9 percent
80-001/ 03L	01/03/80	02/01/80	30 Day LER	Operation in a degraded mode permitted by TS when recirculation low was mismatched by greater than 10 percent for approximately 12 minutes
80-002/ 01T	01/31/80	02/14/80	24 Hours	Error in safety analysis report. Administrative limits will be established to insure compliance with ECCS limits
80-003/ 03L	01/04/80	02/04/80	30 Day LER	Operation in degraded mode as permitted by T.S. when it was on condenser supply line seismic support safety factors may not meet requirements

REVIEWED BY OMPA LOEB: John J. Rizzi **DATE:** 4-10-80

REVIEWED BY NRR DOR: ______ **DATE:** [illegible]

MILLSTONE 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-245
2. Utility Contact: R. Young (203) 447-1792 x475
3. Licensed Thermal Power (MWt): 2011
4. Nameplate Rating (Gross MWe): 735 x 0.9 = 662
5. Design Electrical Rating (Net MWe): 660
6. Maximum Dependable Capacity (Gross MWe): 684
7. Maximum Dependable Capacity (Net MWe): 654
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours In Reporting Period	696.0	1,440.0	81,120.0
12. Hours Reactor Was Critical	696.0	1,440.0	61,825.0
13. Reactor Reserve Shutdown Hours	.0	.0	935.0
14. Hours Generator On-Line	696.0	1,440.0	59,535.9
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,073,204	1,782,656	105,830,601
17. Gross Electrical Energy (MWH)	362,000	591,900	35,823,296
18. Net Electrical Energy (MWH)	346,290	563,155	34,202,937
19. Unit Service Factor	100.0	100.0	73.4
20. Unit Availability Factor	100.0	100.0	73.4
21. Unit Capacity Factor (MDC Net)	76.1	59.8	64.5
22. Unit Capacity Factor (DER Net)	75.4	59.3	63.9
23. Unit Forced Outage Rate	.0	.0	16.3

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	NONE								

SUMMARY

OPERATION WAS AT NEAR FULL POWER AT THE END OF THE REPORT PERIOD.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

MILLSTONE 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Connecticut/New London 5 miles SW of New London
2.	Electric Reliability Council	NPCC
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2700
5.	Design Electrical Rating (MWE Net)	870
6.	Date of Initial Criticality	October 17, 1975
7.	Date of Commercial Operation	December 26, 1975
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Long Island Sound

Utility & Contractor Information

10.	Licensee	Northeast Nuclear Energy Company
11.	Corporate Address	P. O. Box 270 Hartford, Connecticut
12.	Corporate Contact	W. G. Counsil, Vice President
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Combustion Engineering
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	IE Senior Inspector	E. McCabe (FTS: 244-2000)
19.	IE Resident Inspector	J. Shedlosky (203) 447-3179
20.	Licensing Project Manager	E. L. Conner, Jr.
21.	Docket Number	50-336
22.	License Number/Date of Issue	DPR-65, 8/1/75
23.	Public Document Room	Waterford Public Library Rope Ferry Road Route 156 Waterford, Connecticut 06385

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

- The licensee is in the process of replacing the plant charging pumps due to cracking in the pump cylinder heads.

PLANT STATUS:

- With the exception of reactor trips from full power on February 15 and 26, the plant operated at a nominal 100% power.

- LAST IE SITE INSPECTION DATE: 2/29/80

- INSPECTION REPORT NO: 50-336/80-02

SUMMARY

INSPECTION SUMMARY

- 50-336/79-30 - Dec 1-31: Routine, onsite regular and backshift inspection by two resident inspectors (40.5 hours). Areas inspected included the control rooms and the accessible portions of the Enclosure, auxiliary, turbine and intake buildings; the condensate polishing facility; radiation protection; physical security; fire protection; plant operating records; surveillance testing; calibration; maintenance; core power distribution limits; and NRC reporting requirements. Noncompliances: None.

REPORTS RECEIVED FROM LICENSEE

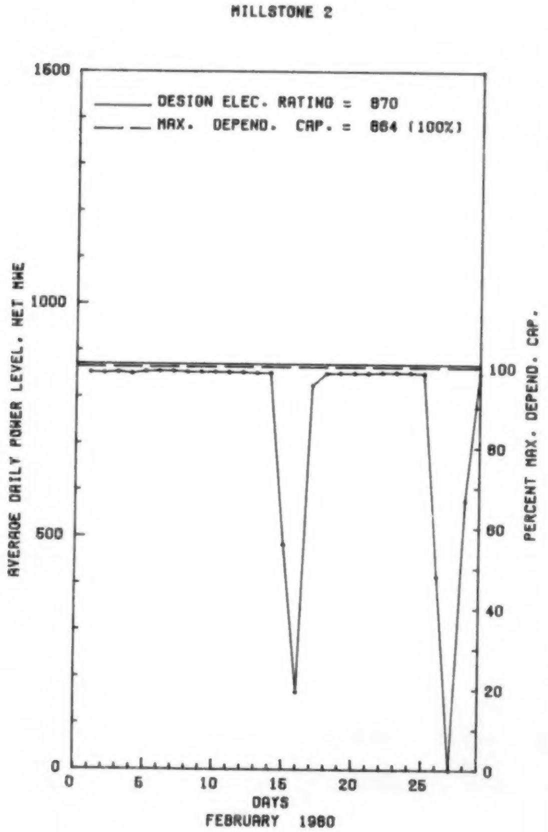
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-001/ INF	12/29/79	01/23/80	Other	Failure of annunciator power supplies due to overload condition
79-034/ 03L	11/03/79	11/20/79	30 Day LER	Excessive leakage identified outer personnel access hatch. Leak repaired and tested
79-035/ 03L	11/02/79	11/28/79	30 Day LER	Channel A high power setpoint found high while reactor was in cold shutdown condition
79-036/ 03L	11/01/79	11/30/79	30 Day LER	Containment integrity degraded for approximately 30 seconds during plant cooldown
79-037/ 03L	11/14/79	12/07/79	30 Day LER	Leakage from boric acid evaporator 'B' concentrates pump discharge line due to hairline crack. Leakage container in the radwaste system
79-038/ 03L	11/13/79	12/13/79	30 Day LER	Leak on 4" radwaste line due to pitting or cracking. Cause under investigation
80-001/ 03L	01/18/80	02/11/80	30 Day LER	Turbine driven auxiliary feedwater pump inoperable due to bearing failure

REVIEWED BY OMPA LOEB: M.R. Beebe DATE: 04/10/80

REVIEWED BY NRR DOR: DATE:

MILLSTONE 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-336
2. Utility Contact: G. H. Howlett (203) 447-1791 x364
3. Licensed Thermal Power (MWt): 2700
4. Nameplate Rating (Gross MWe): 1011 x 0.9 = 910
5. Design Electrical Rating (Net MWe): 870
6. Maximum Dependable Capacity (Gross MWe): 895
7. Maximum Dependable Capacity (Net MWe): 864
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

CAPACITY FACTORS CALCULATED WITH A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	696.0	1,440.0	36,648.0
12. Hours Reactor Was Critical	670.1	1,416.1	26,492.8
13. Reactor Reserve Shutdown Hours	.0	.0	2,072.4
14. Hours Generator On-Line	637.1	1,381.1	25,201.3
15. Unit Reserve Shutdown Hours	.0	.0	335.4
16. Gross Thermal Energy (MWH)	1,683,788	3,684,793	51,383,421
17. Gross Electrical Energy (MWH)	550,474	1,211,436	19,796,081
18. Net Electrical Energy (MWH)	529,350	1,167,061	18,947,319
19. Unit Service Factor	91.5	95.9	68.8
20. Unit Availability Factor	91.5	95.9	69.7
21. Unit Capacity Factor (MDC Net)	88.0	93.8	63.2
22. Unit Capacity Factor (DER Net)	87.4	93.2	61.8
23. Unit Forced Outage Rate	8.5	4.1	22.9

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUELING (CYCLE 4) JULY 1,1980, 9 WEEKS
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWN/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
1	02/15/80	F	23.5	H	3				WHILE PERFORMING REACTOR SYSTEM MATRIX LOGIC & TRIP PATH RELAY TEST, THE REACTOR SCRAMMED. CAUSE OF THE TRIP WAS TRIP CIRCUIT BREAKER TCB-2 OPENING WHILE MATRIX TESTING OTHER BREAKERS.
2	02/26/80	F	16.3	H	3				MAIN GENERATOR LOCKOUT RESULTING IN REACTOR TRIP. CAUSE WAS INADVERTENT ACTUATION OF FAULT TRIP CIRCUITRY DURING TESTING AT THE SITE MAIN SWITCHYARD.
3	02/27/80	F	19.1	A	3		HH	VALVEX	REACTOR TRIPPED ON LOW S/G LEVEL DUE TO FEED REGULATOR VALVE LOCKUP. OVERHAULED FEED REG. VALVE.

SUMMARY

OPERATION WAS AT NEAR FULL POWER DURING THE REPORT PERIOD, EXCEPT FOR THREE REACTOR TRIPS. ON 02/15/80, THE REACTOR SCRAMMED WHEN A TRIP CIRCUIT BREAKER OPENED WHILE MATRIX TESTING OTHER BREAKERS. MAIN GENERATOR LOCKOUT RESULTED IN A REACTOR TRIP ON 02/26/80, FOLLOWING INADVERTENT ACTUATION OF FAULT TRIP CIRCUITRY DURING TESTING AT THE SITE MAIN SWITCHYARD. ON 02/27/80, THE REACTOR TRIPPED ON LOW STEAM GENERATOR LEVEL DUE TO FEED REGULATOR VALVE LOCKUP.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

MONTICELLO

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Minnesota/Wright 30 miles NW of Minneapolis
2.	Electric Reliability Council	MARCA
3.	Type of Reactor	BWR
4.	Power Level (MWT)	1670
5.	Design Electrical Rating (MWE Net)	545
6.	Date of Initial Criticality	December 10, 1970
7.	Date of Commercial Operation	June 30, 1971
8.	Condenser Cooling Method	Once Through with Cooling Tower
9.	Condenser Cooling Water Source	Mississippi River

Utility & Contractor Information

10.	Licensee	Northern States Power Co.
11.	Corporate Address	414 Nicollet Mall - 8th Floor Minneapolis, Minnesota 55401
12.	Corporate Contact	G. H. Neils (Oper.) L. O. Mayer (Lic.)
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	GE
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heisman
18.	IE Senior Inspector	R. F. Warnick
19.	IE Principal Inspector	N. C. Choules
20.	Licensing Project Manager	T. Kevern
21.	Docket Number	50-263
22.	License Number/Date of Issue	DPR-22 issued 9/8/70
23.	Public Document Room	Environmental Conservation Library Minneapolis Public Library 300 Nicollet Mall Minneapolis, Minnesota 55401

INSPECTION STATUS

ENFORCEMENT STATUS

10 CFR 50.59 REQUIRES PROPOSED CHANGES TO THE TECHNICAL SPECIFICATIONS AND UNREVIEWED SAFETY QUESTIONS TO BE SUBMITTED TO AND APPROVED BY THE NRC PRIOR TO IMPLEMENTATION. CONTRARY TO THE ABOVE, THE LICENSEE DID NOT REQUEST NRC APPROVAL PRIOR TO IMPLEMENTATION OF A CHANGE TO THE TECHNICAL SPECIFICATIONS. SPECIFICALLY, ON MAY 1, 1979 THE MAXIMUM AVERAGE PLANAR EXPOSURE OF CYCLE 7 RELOAD 8DB262 FUEL EXCEEDED THE 30,000 MWD/STU FOR WHICH THE CORE WAS ANALYZED IN THE CYCLE 7 RELOAD ANALYSIS SUBMITTED TO THE NRC ON MARCH 21, 1978. ON DECEMBER 21, 1979, THE LICENSEE REQUESTED THAT TECHNICAL SPECIFICATION TABLE 3.1.11.1 (MAPLHGR VS EXPOSURE) BE EXTENDED BEYOND 30,000 MWD/STU. (7920 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

MANAGERIAL ITEMS

- Mr. D. E. Gilberts has been named Vice President - Power Production responsible for all electric generating activities including Nuclear Power Plant Operations. Mr. L. J. Watcher former Vice President - Power Production and System Operation has been named Vice President Power System Operation and Maintenance. (79-20)

PLANT STATUS

- Shut down for refueling.

- LAST IE SITE INSPECTION DATE: January 14, 1980

- INSPECTION REPORT NO: 80-01

SUMMARY

INSPECTION SUMMARY

- Inspection on December 18-20, (79-20): Routine, unannounced inspection of plant operations, nonroutine event followup, and IE Bulletin followup. The inspection involved 21 inspector hours onsite by one NRC inspector. Of the three areas inspected, no items of noncompliance were identified in two areas. One item was identified in one area (exceeding the maximum average linear power exposure).

- Inspection on January 14, (80-01): Licensee actions relative to IE Bulletin No. 79-14, including general discussion on NRC requirements, procedure review, and review of stress calculations and records. The inspection involved eight inspector hours at the consultant's office by one NRC inspector. No items of noncompliance or deviations were identified.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

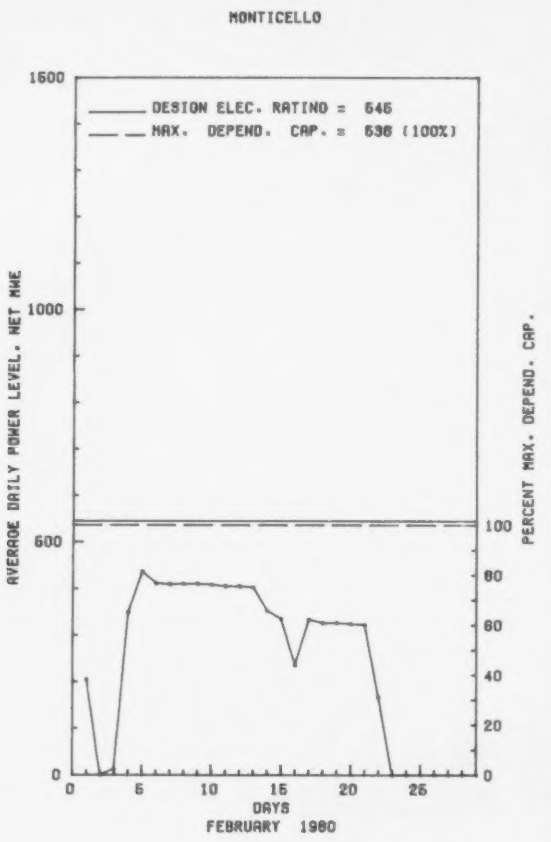
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
80-01/ 03L-0	01/17/80	02/13/80	30 Day LER	Main steam line low pressure switch setpoint drift.

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 4-10-80

REVIEWED BY NRR DOR: **DATE:** 4/14/80

MONTICELLO

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-263
2. Utility Contact: A. L. Myrabo (612) 295-5151
3. Licensed Thermal Power (MWt): 1670
4. Nameplate Rating (Gross MWe): 632 x 0.9 = 569
5. Design Electrical Rating (Net MWe): 545
6. Maximum Dependable Capacity (Gross MWe): 564
7. Maximum Dependable Capacity (Net MWe): 536
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	696.0	1,440.0	75,985.0
12. Hours Reactor Was Critical	479.6	1,223.6	62,754.5
13. Reactor Reserve Shutdown Hours	.0	.0	940.7
14. Hours Generator On-Line	467.1	1,211.1	61,181.2
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	539,513	1,553,878	100,496,740
17. Gross Electrical Energy (MWH)	175,800	521,150	31,499,210
18. Net Electrical Energy (MWH)	166,754	497,080	30,129,284
19. Unit Service Factor	67.1	84.1	80.5
20. Unit Availability Factor	67.1	84.1	80.5
21. Unit Capacity Factor (MDC Net)	44.7	64.4	74.0
22. Unit Capacity Factor (DER Net)	44.0	63.3	72.8
23. Unit Forced Outage Rate	1.7	.7	6.4

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 03/28/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
1	02/01/80	S	45.3	D	1		SD	INSTRU	PRIMARY CONTAINMENT ISOLATION RESET LOGIC MODIFICATION. NUREG 0578, ITEM 2.1.4 IMPLEMENTATION.
2	02/03/80	F	8.2	A	3		IA	INSTRU	FULL IRM SCRAM OCCURRED FROM A SPURIOUS UPSCALE SPIKE ON IRM CHANNEL 14. EARLIER TRIP ON "B" RPS CHANNEL WAS NOT FULLY RESET.
3	02/16/80	S	.0	B	5		MC	HTEXCH	REDUCED POWER TO 30% TO PERFORM CONDENSER TUBE LEAK TEST.
4	02/22/80	S	175.4	C	1		ZZ	ZZZZZZ	END OF CYCLE 7 REFUELING OUTAGE.

SUMMARY

THE UNIT OPERATED IN AN END-OF-CYCLE COASTDOWN, WITH A SHUTDOWN ON 02/02/80 TO COMPLETE TMI RELATED MODIFICATIONS, A LOAD REDUCTION ON 02/14/80 DUE TO BEARING FAILURE IN A FEEDWATER PUMP, AND A REDUCTION ON 02/16/80 FOR MAIN CONDENSER TUBE LEAK INSPECTION. ON 02/22/80, THE UNIT WAS SHUTDOWN FOR REFUELING.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

NINE MILE POINT 1

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	New York/Oswego 8 miles NE of Oswego
2.	Electric Reliability Council	NPCC
3.	Type of Reactor	BWR
4.	Power Level (MWT)	1850
5.	Design Electrical Rating (MWE Net)	620
6.	Date of Initial Criticality	September 5, 1969
7.	Date of Commercial Operation	December 1969
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Lake Ontario

Utility & Contractor Information

10.	Licensee	Niagara Mohawk Power Corporation
11.	Corporate Address	300 Erie Boulevard West Syracuse, New York 13202
12.	Corporate Contact	R. Schneider, Vice President Electric Operations
13.	Architect/Engineer	Utility
14.	Nuclear Steam Supply System	GE
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Senior Inspector	E. McCabe
19.	IE Principal Inspector	W. Baunack
20.	Licensing Project Manager	P. Polk
21.	Docket Number	50-220
22.	License Number/Date of Issue	DPR-63, 12/26/74
23.	Public Document Room	State University College of Oswego Penfield Library - Documents Oswego, NY 13126 (315)341-2323

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

MANAGERIAL ITEMS:

+ Mr. T. Lempges, General Superintendent, Nuclear Generation, has been promoted to Vice President, Nuclear Generation. He replaces Mr. R. R. Schneider who is retiring.

PLANT STATUS:

+ Operating at 90% power, preconditioning fuel following startup on 3/9. The unit was shutdown 3/2 to 3/9 to complete Short Term Lessons Learned Modifications and other maintenance. Representative stack and offgas release rates are 20 and 10,000 microcuries per second, respectively.

+ LAST IE SITE INSPECTION DATE: 3/3-7/80

+ INSPECTION REPORT NO: 50-220/80-02

SUMMARY

INSPECTION SUMMARY

+ 50-220/79-26 - Dec 10, 11 & 13: Special, unannounced inspection by a region-based inspector (19 hours) of the licensee's response to IE Bulletin 79-19 including licensee action on IE Circulars, documents, procedures, training, audits, facility tours, and records. Noncompliances: None.

+ 50-220/80-01 - Feb 4-8: Routine, unannounced inspection by two region-based inspectors (6[illegible] hours) of logs and operating records; facility tours, and control room operations. Noncompliances: One (failure to maintain a valve locked as required).

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

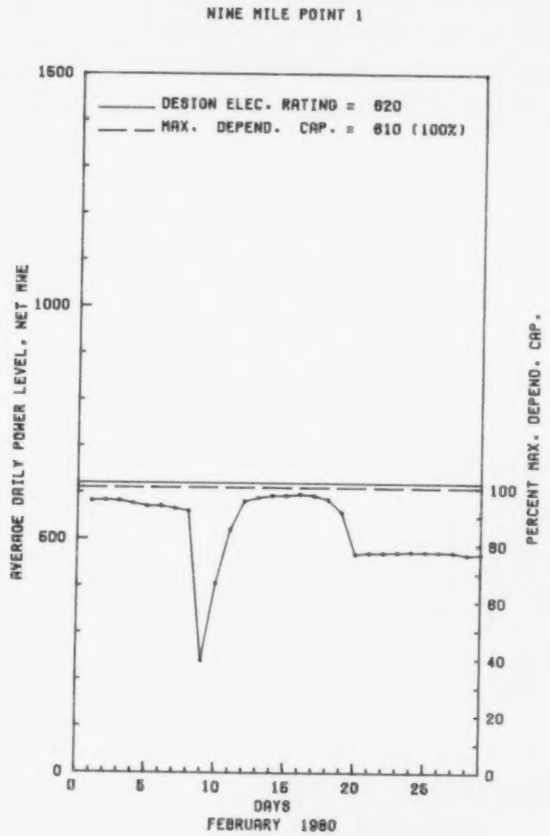
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
80-004/ 03L	01/29/80	02/05/80	30 Day LER	Average planar linear heat generation rate exceeded limit
80-005/ 01X	02/03/80	02/15/80	30 Day LER	Reactor vessel triple low level setpoint determined to be calculated in a nonconservative direction
80-006/ 04L	01/24/80	02/20/80	Environmental Deviation	Exceeded condenser delta temperature limit

REVIEWED BY OMPA LOEB: John J. Rizzi **DATE:** 4-10 80

REVIEWED BY NRR DOR: Phillip J. Polk **DATE:** 4/4/80

NINE MILE POINT 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-220
2. Utility Contact: B. Hawksley (315) 343-2110 x1312
3. Licensed Thermal Power (MWt): 1850
4. Nameplate Rating (Gross MWe): 755 x 0.85 = 642
5. Design Electrical Rating (Net MWe): 620
6. Maximum Dependable Capacity (Gross MWe): 630
7. Maximum Dependable Capacity (Net MWe): 610
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	696.0	1,440.0	90,552.0
12. Hours Reactor Was Critical	686.0	1,430.0	66,782.7
13. Reactor Reserve Shutdown Hours	.0	.0	1,204.2
14. Hours Generator On-Line	679.0	1,423.0	64,233.5
15. Unit Reserve Shutdown Hours	.0	.0	20.2
16. Gross Thermal Energy (MWH)	1,110,385	2,437,657	104,462,258
17. Gross Electrical Energy (MWH)	371,911	822,894	34,332,484
18. Net Electrical Energy (MWH)	359,140	795,493	33,245,258
19. Unit Service Factor	97.6	98.8	70.9
20. Unit Availability Factor	97.6	98.8	71.0
21. Unit Capacity Factor (MDC Net)	84.6	90.6	60.2
22. Unit Capacity Factor (DER Net)	83.2	89.1	59.2
23. Unit Forced Outage Rate	.0	.0	9.1

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-7	02/09/80	S	17.0	B	1				CORE SPRAY I.V. QUARTERLY TEST.
80-8	02/19/80	F	.0	A	5				REDUCTION TO 77% POWER DUE TO FAILURE OF #125 (HIGH PRES.) FEEDWATER HEATER.

SUMMARY

OPERATION WAS ROUTINE, WITH A REDUCTION TO 77% POWER FROM 02/19/80 THROUGH THE END OF THE REPORT PERIOD DUE TO HIGH PRESSURE FEEDWATER HEATER FAILURE. A LOAD REDUCTION ALSO OCCURRED ON 02/09/80 FOR TESTING.

LEGEND

[1] Type:
F–Forced
S–Scheduled

[2] Reason:
A–Equipment Failure
B–Maintenance or Test
C–Refueling
D–Regulatory Restriction
E–Operator Training & License Examination
F–Administrative
G–Operational Error
H–Other

[3] Method:
1–Manual
2–Manual Scram
3–Automatic Scram
4–Continuations
5–Load Reduction
9–Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

NORTH ANNA 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Louisa County, Virginia 40 Miles NW of Richmond, Virginia
2.	Electric Reliability Council	Southeastern
3.	Type of Reactor	PWR
4.	License Power Level (MWT)	2775
5.	Design Electrical Rating (MWE Net)	907
6.	Date of Initial Criticality	April 5, 1978
7.	Date of Commercial Operation	June 6, 1978
8.	Condenser Cooling Method	Once through
9.	Condenser Cooling Water Source	Lake Anna

Utility & Contractor Information

10.	Licensee	Virginia Electric & Power Co.
11.	Corporate Address	P. O. Box 26666 Richmond, Virginia 23261
12.	Corporate Contact	J. H. Ferguson Exec. V.P. - Power
13.	Architect/Engineer	Stone & Webster
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. Lewis
18.	IE Senior Inspector	P. J. Kellog
19.	IE Resident Inspector	M. S. Kidd (703) 894-6927
20.	Licensing Project Manager	L. Engle
21.	Docket Number	50-338
22.	License Number/Date of Issue	NPF-4, 11/26/77
23.	Public Document Room	Alderman Library/Manuscripts Dept. Univ. of Virginia/Charlottesville VA 22901 & Louisa County Courthouse, Louisa, VA 23093

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO 10 CFR 50, APPENDIX B, CRITERION V, THE ACTIVITIES AFFECTING QUALITY WERE NOT ACCOMPLISHED IN ACCORDANCE WITH THE INSTRUCTIONS IN THAT NO USAGE INFORMATION HAD BEEN ESTABLISHED AND MAINTAINED FOR FOUR TORQUE WRENCHES (NQC 561, 526, 585, AND 586) WHICH WERE IN THE MEASURING AND TEST EQUIPMENT CALIBRATION PROGRAM AND ALSO A COMPLETE HISTORY RECORD FOR EACH DEVICE WAS NOT MAINTAINED.
(7949 DEFICIENCY)

CONTRARY TO 10 CFR 50, APPENDIX B, CRITERION V, THE ACTIVITIES AFFECTING QUALITY WERE NOT ACCOMPLISHED IN ACCORDANCE WITH THE INSTRUCTIONS. VARIOUS CALCULATIONAL ERRORS/MISTAKES WERE CONTAINED IN THE DATA SHEETS OF THE FIVE DIFFERENT PT'S THROUGHOUT PT-11 (CORE REACTIVITY BALANCE) WHICH WAS PERFORMED ON 2/22/79, PT-20.2 (AXIAL FLUX DIFFERENCE) WHICH WAS COMPLETED ON 3/1/79, PT-31.1.1 (DELTA T/T AVG PROTECTION CHANNEL 1 FUNCTIONAL TEST - T412) WHICH WAS PERFORMED ON 6/14/79, PT-11 WHICH WAS COMPLETED ON 6/18/79, AND PT-24 (CALORIMETRIC) WHICH WAS PERFORMED ON 9/21/79. ALTHOUGH THESE DOCUMENTS HAD RECEIVED TWO LEVELS OF REVIEW, THE ERRORS/MISTAKES HAD NOT BEEN IDENTIFIED.
(7949 INFRACTION)

CONTRARY TO THE TECHNICAL SPECIFICATIONS 4.0.2.A AND 4.7.14.1.1.C, NO SURVEILLANCE TEST WAS PERFORMED FOR PT-100.3 (FIRE SUPPRESSION WATER SYSTEM - VALVE POSITION VERIFICATION) FOR ABOUT 61 DAYS BETWEEN 8/12/79 AND 10/13/79.
(7949 INFRACTION)

CONTRARY TO TECHNICAL SPECIFICATION 6.8.1.C, THE REQUIRED INSTRUCTIONS OF PT 75.2B WERE NOT IMPLEMENTED IN STEP 4.7.A OF THE PROCEDURE WHICH WAS PERFORMED ON 8/24/79, AND ALSO IN STEP 4.1 OF THE PROCEDURE WHICH WAS PERFORMED ON 5/24/79, IN THAT THE FREQUENCY WAS NOT DOUBLED IN THE FOLLOWING MONTH AND THE FLOW WAS NOT RECORDED IN THE STEP 4.1.
(7949 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

- UNIT IS OPERATING AT 100% POWER.
- LAST IE SITE INSPECTION DATE: FEBRUARY 29, 1980
- INSPECTION REPORT NO: 50-338/80-11

SUMMARY

INSPECTION SUMMARY

- INSPECTION JANUARY 29 - FEBRUARY 1 (80-01): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 28 INSPECTOR-HOURS ON SITE IN THE AREAS OF NON-RADIOLOGICAL ENVIRONMENTAL PROTECTION REQUIREMENTS INCLUDING WATER QUALITY MONITORING; BIOLOGICAL SURVEILLANCE; GENERAL ECOLOGICAL SURVEY; MANAGEMENT CONTROLS. OF THE FOUR AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION JANUARY 28-31 (80-02): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 32 INSPECTOR-HOURS ON SITE IN THE AREAS OF SOLID WASTE PACKAGING AND TRANSPORTATION, FACILITY RESPONSE TO IE BULLETIN 79-19, AND FOLLOWUP OF WORKER CONCERNS. NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
80-001/01T-0	01/20/80	02/01/80	14 DAY LER	EXCESS LETDOWN LINES OVERSTRESSED DUE TO INCORRECT VALVE WEIGHT.
80-002/01T-0	01/08/80	01/18/80	14 DAY LER	PIPE HANGER STRESS EXCEEDS ALLOWABLE DUE TO VALVE WEIGHT ERROR.
80-008/01T-0	01/15/80	01/25/80	14 DAY LER	CONTROL ROOM AIR CONDITION CHILLER CONTROL PANEL NOT SEISMICALLY QUALIFIED.
80-010/01T-0	01/15/80	01/25/80	14 DAY LER	PIPE HANGERS IN SERVICE WATER SCREEN WASH SYSTEM OVERSTRESSED DUE TO ANALYSIS WITH INCORRECT VALVE WEIGHTS.

INSPECTION JANUARY 21-25 (80-06): INCLUDED REVIEW OF THE LICENSEE'S SECURITY PLAN, SECURITY PROGRAM AUDIT, SECURITY ORGANIZATION, VITAL AND PROTECTED AREA BARRIERS, SECURITY LIGHTING, ACCESS CONTROLS, ALARM STATIONS, DETECTION AND ASSESSMENT AIDS, COMMUNICATIONS, TESTING/MAINTENANCE OF SECURITY EQUIPMENT, COMPENSATORY MEASURES, ALARM RESPONSES, LOCKS/KEYS AND SECURITY RELATED RECORDS AND REPORTS. THE INSPECTION INVOLVED 62 INSPECTOR-HOURS ON SITE BY TWO NRC INSPECTORS. THE INSPECTION WAS BEGUN DURING AN OFFSHIFT PERIOD; 24 INSPECTION-HOURS WERE ACCOMPLISHED DURING OFFSHIFT PERIODS. THE LICENSEE WAS FOUND TO BE IN COMPLIANCE WITH NRC REQUIREMENTS WITHIN THE AREAS EXAMINED DURING THE INSPECTION EXCEPT FOR THE FOLLOWING ITEM: SECURITY ORGANIZATION - A WRITTEN SECURITY PROCEDURE DETAILING THE DUTIES OF INDIVIDUALS RESPONSIBLE FOR TESTING SPECIAL DETECTION EQUIPMENT WAS NOT ESTABLISHED. CORRECTIVE ACTION WAS COMPLETED AND INSPECTED ON JANUARY 24, 1980. NO RESPONSE REQUIRED.

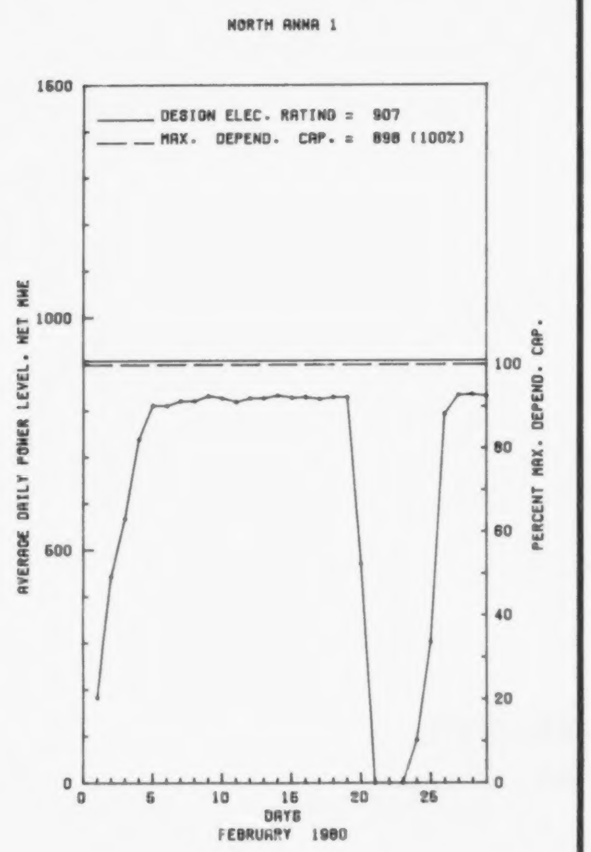
INSPECTION FEBRUARY 6-8 (80-08): THIS ROUTINE ANNOUNCED INSPECTION INVOLVED 16 INSPECTOR-HOURS ON SITE IN THE AREAS OF IE BULLETIN 79-14, "SEISMIC ANALYSIS FOR AS-BUILT SAFETY-RELATED PIPING SYSTEMS." NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

REVIEWED BY OMPA LOEB: T.C. Cintula **DATE:** 4·10·80

REVIEWED BY NRR DOR: L Engle **DATE:** 4·11·80

NORTH ANNA 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-338
2. Utility Contact: W. R. Madison (703) 894-5151
3. Licensed Thermal Power (MWt): 2775
4. Nameplate Rating (Gross MWe): 1089 x 0.9 = 980
5. Design Electrical Rating (Net MWe): 907
6. Maximum Dependable Capacity (Gross MWe): 928
7. Maximum Dependable Capacity (Net MWe): 898
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	696.0	1,440.0	15,217.0
12. Hours Reactor Was Critical	604.5	947.9	11,113.1
13. Reactor Reserve Shutdown Hours	7.0	37.0	51.6
14. Hours Generator On-Line	595.9	798.7	10,851.8
15. Unit Reserve Shutdown Hours	0	0	0
16. Gross Thermal Energy (MWH)	1,690,114	1,896,810	28,176,723
17. Gross Electrical Energy (MWH)	471,315	524,079	8,874,478
18. Net Electrical Energy (MWH)	442,506	486,974	8,340,470
19. Unit Service Factor	85.6	55.5	71.3
20. Unit Availability Factor	85.6	55.5	71.3
21. Unit Capacity Factor (MDC Net)	70.8	37.7	61.5
22. Unit Capacity Factor (DER Net)	70.1	37.3	60.4
23. Unit Forced Outage Rate	14.4	12.0	10.6

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-08	02/01/80	F	4.0	B	3				FEEDWATER SYSTEM FLOW OSCILLATION SET UP WHILE TESTING OF "B" STEAM GENERATOR LEVEL CONTROL SYSTEM CAUSED "A" STEAM GENERATOR TO GO HI. REACTOR TRIPPED FROM HI-HI LEVEL "A" STM. GEN.
80-09	02/20/80	F	93.4	H	3				HIGH NEGATIVE FLUX RATE TRIP-ALL 4 POWER RANGE CHANNELS. A CONTRACT LABORER CAUGHT HIS CLOTHING ON THE MOTOR BREAKER FOR THE CONTROL ROD DRIVE MOTOR GEN. SET CAUSING IT TO OPEN. THIS LOSS OF POWER TO THE CONTROL RODS CAUSED THEM TO FALL INTO THE CORE. AS THE ROD(S) ACCELERATED, A NEGATIVE FLUX RATE WAS SET UP CAUSING THE REACTOR TRIP.
80-10	02/24/80	F	2.7	G	3				LO-LO LEVEL "B" STEAM GENERATOR. WHILE RESTARTING THE UNIT, OPERATOR INEXPERIENCE.

SUMMARY

THE UNIT EXPERIENCED THREE TRIPS DURING THE REPORT PERIOD. ON 02/01/80, THE REACTOR TRIPPED WHEN FEEDWATER SYSTEM FLOW OSCILLATION DURING TESTING OF THE "B" STEAM GENERATOR LEVEL CONTROL SYSTEM CAUSED THE "A" STEAM GENERATOR LEVEL TO GO HI. ON 02/20/80, AN EMPLOYEE CAUGHT HIS CLOTHING ON THE REACTOR BREAKER FOR THE CONTROL ROD DRIVE MOTOR GENERATOR SET, CAUSING IT TO OPEN. AS THE CONTROL RODS FELL INTO THE CORE A NEGATIVE FLUX RATE WAS SET UP, CAUSING THE REACTOR TRIP. DURING RESTART ON 02/24/80, THE UNIT TRIPPED ON LO-LO LEVEL IN THE "B" STEAM GENERATOR.

LEGEND

[1] Type:
F–Forced
S–Scheduled

[2] Reason:
A–Equipment Failure
B–Maintenance or Test
C–Refueling
D–Regulatory Restriction
E–Operator Training & License Examination
F–Administrative
G–Operational Error
H–Other

[3] Method:
1–Manual
2–Manual Scram
3–Automatic Scram
4–Continuations
5–Load Reduction
9–Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

OCONEE 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	South Carolina/Oconee 30 miles W of Greenville
2.	Electric Reliability Council	Southeastern
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2568
5.	Design Electrical Rating (MWE Net)	887
6.	Date of Initial Criticality	April 19, 1973
7.	Date of Commercial Operation	July 15, 1973
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Keowee Lake

Utility & Contractor Information

10.	Licensee	Duke Power Company
11.	Corporate Address	422 S. Church Street Charlotte, North Carolina 28242
12.	Corporate Contact	W. O. Parker, Jr.
13.	Architect/Engineer	DPC and Bechtel (Containment Bldg.)
14.	Nuclear Steam Supply System	B&W
15.	Constructor	DPC

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R.C. Lewis
18.	IE Senior Inspector	E.H. Verdery
19.	IE Resident Inspector	F. Jape (803) 882-6927
20.	Licensing Project Manager	G. Rivenbark
21.	Docket Number	50-269
22.	License Number/Date of Issue	DPR-38, 2/6/73
23.	Public Document Room	Oconee County Library 201 S. Spring Street Walhalla, South Carolina 29691

INSPECTION STATUS

ENFORCEMENT STATUS

AS REQUIRED BY 10 CFR 20.203(F), EACH CONTAINER OF LICENSED MATERIAL SHALL BEAR A DURABLE, CLEARLY VISIBLE LABEL IDENTIFYING THE RADIOACTIVE CONTENTS AND PROVIDING SUFFICIENT INFORMATION TO PERMIT INDIVIDUALS HANDLING OR USING THE CONTAINERS, OR WORKING IN THE VICINITY THEREOF TO TAKE PRECAUTIONS TO AVOID OR MINIMIZE EXPOSURES. ON NOVEMBER 27, 1979, FIVE BAGS OF RADIOACTIVE MATERIAL LOCATED ON THE UNIT 3 LOW PRESSURE INJECTION DECK, WERE OBSERVED WITHOUT THE PROPER LABELS.
(7935 DEFICIENCY)

AS REQUIRED BY TECHNICAL SPECIFICATION 6.4, THE STATION SHALL BE OPERATED AND MAINTAINED IN ACCORDANCE WITH APPROVED PROCEDURES. (1) STATION PROCEDURE HP/0/B/1000/10, "WORKING LIMITS FOR CONTAMINATION CONTROL" STATES THAT IN AREAS WHERE CONTAMINATION EXISTS ABOVE THE WORKING LIMITS SPECIFIED, A POSTED AND/OR ROPED-OFF CONTAMINATION AREA WILL BE ESTABLISHED. PROCEDURE HP/0/B/1000/10 WAS NOT FOLLOWED IN THAT A POSTED AND/OR ROPED-OFF CONTAMINATION AREA WAS NOT ESTABLISHED ON NOVEMBER 19, 1979 AROUND FOUR UNIT 1 HIGH PRESSURE TURBINE HOLDOWN BOLTS CONTAMINATED ABOVE THE LIMITS SPECIFIED BY PROCEDURE. (2) STATION PROCEDURE OP/0/A/1102/06, "REMOVAL AND RESTORATION OF STATION EQUIPMENT", ENCLOSURE 7.1, STEP 7.1.13 SPECIFIES THE PROPER VALVE OR SPEAKER POSITION REQUIRED FOR RESTORATION OF A SYSTEM TO SERVICE. ON OCTOBER 16, 1979, WHILE RESTORING THE UNIT 3 LOW PRESSURE INJECTION SYSTEM "B" TRAIN TO SERVICE, VENT VALVE EGWD-152 WAS NOT CLOSED AS CALLED FOR ON THE REMOVAL AND RESTORATION OF STATION EQUIPMENT CHECKLIST, ALTHOUGH THE CHECKLIST WAS INSTALLED AND DATED INDICATING THE VALVE WAS LEFT IN THE PROPER POSITION.
(7935 INFRACTION)

CONTRARY TO THE QA PLAN, ACCEPTED BY NRC JUNE 29, 1978 IN WHICH DPC COMMITTED TO RG 1.39, ON JANUARY 30 AND 31, 1980 COMMITMENTS TO ADHERE TO STATED HOUSEKEEPING PRACTICES WITHIN PLANT AREAS OCCUPIED BY SAFETY RELATED EQUIPMENT WERE NOT FOLLOWED. SPECIFICALLY THE LOW PRESSURE INJECTION, BUILDING SPRAY AND HIGH PRESSURE INJECTION ROOMS CONTAINED EXCESS MATERIALS, EQUIPMENT AND DEBRIS WHICH APPARENTLY REMAINED FROM PREVIOUS JOB ACTIVITIES IN THESE LOCATIONS.
(8005 DEVIATION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

+ RETURNED TO SERVICE 2/25/80 FOLLOWING REFUELING OUTAGE.

+ LAST IE SITE INSPECTION DATE: FEBRUARY 27-29, 1980

+ INSPECTION REPORT NO: 50-269/80-05

SUMMARY

INSPECTION SUMMARY

+ INSPECTION NOVEMBER 11 AND 15-16 (79-37): THIS SPECIAL, ANNOUNCED INSPECTION INVOLVED 36 INSPECTOR-HOURS ON SITE IN THE AREAS OF TRANSIENT ANALYSIS AND REPORTABILITY REQUIREMENTS. OF THE TWO AREAS INSPECTED, TWO ITEMS OF NONCOMPLIANCE WERE FOUND IN ONE AREA (INFRACTION - INADEQUATE PROCEDURES TO CORRECT SPECIFIC AND FORESEEN POTENTIAL MALFUNCTIONS; AND INFRACTION - EXCESSIVE REACTOR COOLANT SYSTEM COOLDOWN RATE; ONE APPARENT DEVIATION WAS FOUND IN ONE AREA (FAILURE TO NOTIFY NRC WITHIN ONE HOUR WHEN UNIT 3 WAS IN AN UNEXPECTED OPERATING CONDITION).

INSPECTION JANUARY 21-25 AND JANUARY 28 - FEBRUARY 1 (80-04): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 18 INSPECTOR-HOURS ON SITE IN THE AREAS OF COORDINATION WITH OFFSITE SUPPORT AGENCIES, EMERGENCY FACILITIES, EQUIPMENT AND PROCEDURES, MEANS FOR DETERMINING A RELEASE, EMERGENCY TRAINING, EMERGENCY DRILLS, AND EMERGENCY ORGANIZATION. OF THE SIX AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION JANUARY 7-31 (80-05): THIS ROUTINE INSPECTION INVOLVED 20 INSPECTOR-HOURS ON SITE IN THE AREAS OF PLANT OPERATIONS, OUTAGE ACTIVITIES, HOUSEKEEPING, FOLLOWUP OF IEB'S, IEC'S AND IE IN'S, PLANT TOURS AND WITNESSING A TEST. OF THE SIX AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN FIVE AREAS; ONE APPARENT DEVIATION WAS FOUND IN ONE AREA (INADEQUATE HOUSEKEEPING).

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

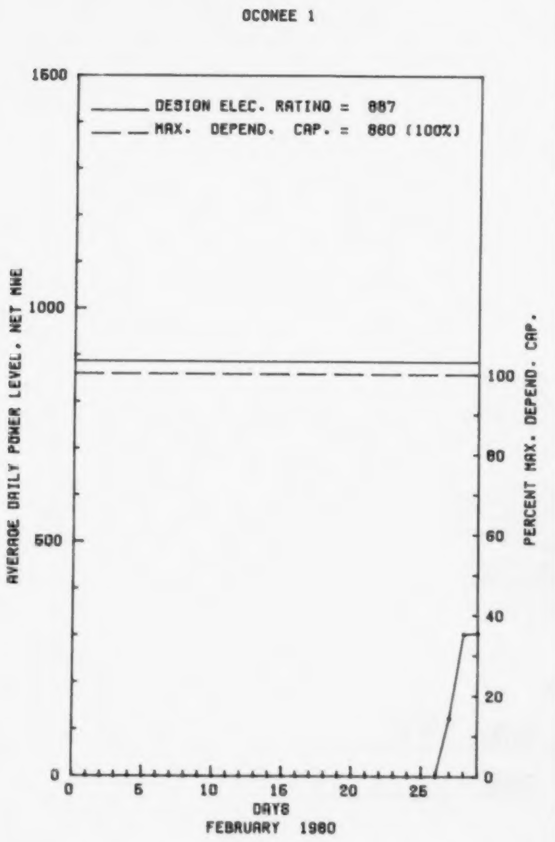
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
80-001/ 011-0	01/08/80	01/22/80	14 DAY LER	DOSES FOR GASEOUS ACTIVITY IN CONTROL ROOM FOLLOWING MAXIMUM HYPOTHETICAL ACCIDENT NOT EVALUATED PER CRITERION 11 FSAR SECTION 1A.

REVIEWED BY OMPA LOEB: ______ **DATE:** 4/10/80

REVIEWED BY NRR DOR: ______ **DATE:** 4/11/80

OCONEE 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-269
2. Utility Contact: J. A. Reavis (704) 373-8552
3. Licensed Thermal Power (MWt): 2568
4. Nameplate Rating (Gross MWe): 1038 x 0.9 = 934
5. Design Electrical Rating (Net MWe): 887
6. Maximum Dependable Capacity (Gross MWe): 899
7. Maximum Dependable Capacity (Net MWe): 860
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

CAPACITY FACTORS CALCULATED WITH A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	696.0	1,440.0	58,081.0
12. Hours Reactor Was Critical	114.5	114.5	40,629.2
13. Reactor Reserve Shutdown Hours	.0	.0	0
14. Hours Generator On-Line	62.8	62.8	38,009.9
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	54,670	54,670	89,272,789
17. Gross Electrical Energy (MWH)	19,820	19,820	30,934,120
18. Net Electrical Energy (MWH)	9,277	6,961	29,238,460
19. Unit Service Factor	9.0	4.4	65.4
20. Unit Availability Factor	9.0	4.4	65.4
21. Unit Capacity Factor (MDC Net)	1.5	.6	583
22. Unit Capacity Factor (DER Net)	1.5	.5	56.8
23. Unit Forced Outage Rate	83.2	83.2	18.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
1	01/16/80	S	209.2	D	4		XX	PIPEXX	COMPLETION OF PIPE HANGER/SUPPORT INSPECTION AND MODIFICATION (IE BULLETIN 79-02 AND 79-14)
2	02/09/80	F	311.3	A	9		CC	HTEXCH	"A" STEAM GENERATOR MANWAY GASKETS REPLACED.
3	02/22/80	S	94.5	B	9		ZZ	ZZZZZZ	WATER CHEMISTRY OUT OF LIMITS.
4	02/26/80	S	18.2	E	9		ZZ	ZZZZZZ	OPERATOR TRAINING
5	02/27/80	F	.0	A	5		IA	INSTRU	HOLDING AT 20% POWER TO CORRECT ICS (CONTROL) PROBLEM.
6	02/27/80	S	.0	B	5		ZZ	ZZZZZZ	HOLDING AT 40% POWER FOR POWER ESCALATION TESTING.

SUMMARY

THE UNIT RETURNED TO SERVICE ON 02/27/80 FROM A REFUELING OUTAGE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

OCONEE 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	South Carolina/Oconee 30 miles W of Greenville
2.	Electric Reliability Council	Southeastern
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2568
5.	Design Electrical Rating (MWE Net)	887
6.	Date of Initial Criticality	November 11, 1973
7.	Date of Commercial Operation	September 9, 1974
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Keowee Lake

Utility & Contractor Information

10.	Licensee	Duke Power Company
11.	Corporate Address	422 S. Church Street Charlotte, North Carolina 28242
12.	Corporate Contact	W. O. Parker, Jr.
13.	Architect/Engineer	DPC/Bechtel
14.	Nuclear Steam Supply System	B&W
15.	Constructor	DPC

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R.C. Lewis
18.	IE Senior Inspector	E.H. Verdery
19.	IE Resident Inspector	F. Jape (803) 882-6927
20.	Licensing Project Manager	G. Rivenbark
21.	Docket Number	50-270
22.	License Number/Date of Issue	DPR-47, 10/6/73
23.	Public Document Room	Oconee County Library 201 S. Spring Street Walhalla, South Carolina 29691

INSPECTION STATUS

ENFORCEMENT STATUS

AS REQUIRED BY 10 CFR 20.203(F), EACH CONTAINER OF LICENSED MATERIAL SHALL BEAR A DURABLE, CLEARLY VISIBLE LABEL IDENTIFYING THE RADIOACTIVE CONTENTS AND PROVIDING SUFFICIENT INFORMATION TO PERMIT INDIVIDUALS HANDLING OR USING THE CONTAINERS, OR WORKING IN THE VICINITY THEREOF TO TAKE PRECAUTIONS TO AVOID OR MINIMIZE EXPOSURES. ON NOVEMBER 27, 1979, FIVE BAGS OF RADIOACTIVE MATERIAL LOCATED ON THE UNIT 3 LOW PRESSURE INJECTION DECK, WERE OBSERVED WITHOUT THE PROPER LABELS.
(7932 DEFICIENCY)

AS REQUIRED BY TECHNICAL SPECIFICATION 6.4, THE STATION SHALL BE OPERATED AND MAINTAINED IN ACCORDANCE WITH APPROVED PROCEDURES. (1) STATION PROCEDURE HP/0/B/1000/10, "WORKING LIMITS FOR CONTAMINATION CONTROL" STATES THAT IN AREAS WHERE CONTAMINATION EXISTS ABOVE THE WORKING LIMITS SPECIFIED, A POSTED AND/OR ROPED-OFF CONTAMINATION AREA WILL BE ESTABLISHED. PROCEDURE HP/0/B/1000/10 WAS NOT FOLLOWED IN THAT A POSTED AND/OR ROPED-OFF CONTAMINATION AREA WAS NOT ESTABLISHED ON NOVEMBER 19, 1979 AROUND FOUR UNIT 1 HIGH PRESSURE TURBINE HOLDOWN BOLTS CONTAMINATED ABOVE THE LIMITS SPECIFIED BY PROCEDURE. (2) STATION PROCEDURE OP/0/A/1102/06, "REMOVAL AND RESTORATION OF STATION EQUIPMENT", ENCLOSURE 7.1, STEP 7.1.13 SPECIFIES THE PROPER VALVE OR SPEAKER POSITION REQUIRED FOR RESTORATION OF A SYSTEM TO SERVICE. ON OCTOBER 16, 1979, WHILE RESTORING THE UNIT 3 LOW PRESSURE INJECTION SYSTEM "B" TRAIN TO SERVICE, VENT VALVE EGWD-152 WAS NOT CLOSED AS CALLED FOR ON THE REMOVAL AND RESTORATION OF STATION EQUIPMENT CHECKLIST, ALTHOUGH THE CHECKLIST WAS INSTALLED AND DATED INDICATING THE VALVE WAS LEFT IN THE PROPER POSITION.
(7932 INFRACTION)

TECHNICAL SPECIFICATION 6.4.1 E REQUIRES PREVENTIVE OR CORRECTIVE MAINTENANCE TO BE ACCOMPLISHED BY ADHERING TO APPROVED WRITTEN PROCEDURES. ON DECEMBER 9, 1979 AN I&E TECHNICIAN PERFORMED MAINTENANCE ON TWO UNIT 2 ENGINEERED SAFEGUARDS VALVES WHILE THE UNIT WAS AT 99% FULL POWER WHEREAS HIS ASSIGNED TASK WAS TO PERFORM MAINTENANCE ON THESE SAME VALVES FOR UNIT 1, WHICH WAS IN A REFUELING OUTAGE.
(7939 INFRACTION)

CONTRARY TO THE QA PLAN, ACCEPTED BY NRC JUNE 29, 1978 IN WHICH DPC COMMITTED TO RG 1.39, ON JANUARY 30 AND 31, 1980 COMMITMENTS TO ADHERE TO STATED HOUSEKEEPING PRACTICES WITHIN PLANT AREAS OCCUPIED BY SAFETY RELATED EQUIPMENT WERE NOT FOLLOWED. SPECIFICALLY THE LOW PRESSURE INJECTION, BUILDING SPRAY AND HIGH PRESSURE INJECTION ROOMS CONTAINED EXCESS MATERIALS, EQUIPMENT AND DEBRIS WHICH APPARENTLY REMAINED FROM PREVIOUS JOB ACTIVITIES IN THESE LOCATIONS.
(8004 DEVIATION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

+ SHUTDOWN 3/5/80 FOR 90 DAY OUTAGE FOR REFUELING, SEISMIC AND FIRE PROTECTION MODIFICATIONS. MODIFICATIONS REQUIRED BY NUREG 0578.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

+ REFUELING OUTAGE STARTING 3/5/80

+ LAST IE SITE INSPECTION DATE: FEBRUARY 25-29, 1980

+ INSPECTION REPORT NO: 50-270/80-04

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-008/ 03L-0	11/21/79	01/07/80	30 DAY LER	INOPERABLE REACTOR BUILDING PIPE PENETRATION.

SUMMARY

INSPECTION SUMMARY

+ INSPECTION NOVEMBER 11 AND 15-16 (79-34): THIS SPECIAL, ANNOUNCED INSPECTION INVOLVED 36 INSPECTOR-HOURS ON SITE IN THE AREAS OF TRANSIENT ANALYSIS AND REPORTABILITY REQUIREMENTS. OF THE TWO AREAS INSPECTED, TWO ITEMS OF NONCOMPLIANCE WERE FOUND IN ONE AREA (INFRACTION - INADEQUATE PROCEDURES TO CORRECT SPECIFIC AND FORESEEN POTENTIAL MALFUNCTIONS; AND INFRACTION - EXCESSIVE REACTOR COOLANT SYSTEM COOLDOWN RATE; ONE APPARENT DEVIATION WAS FOUND IN ONE AREA (FAILURE TO NOTIFY NRC WITHIN ONE HOUR WHEN UNIT 3 WAS IN AN UNEXPECTED OPERATING CONDITION).

INSPECTION JANUARY 21-25 AND JANUARY 28 - FEBRUARY 1 (80-03): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 18 INSPECTOR-HOURS ON SITE IN THE AREAS OF COORDINATION WITH OFFSITE SUPPORT AGENCIES, EMERGENCY FACILITIES, EQUIPMENT AND PROCEDURES, MEANS FOR DETERMINING A RELEASE, EMERGENCY TRAINING, EMERGENCY DRILLS, AND EMERGENCY ORGANIZATION. OF THE SIX AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

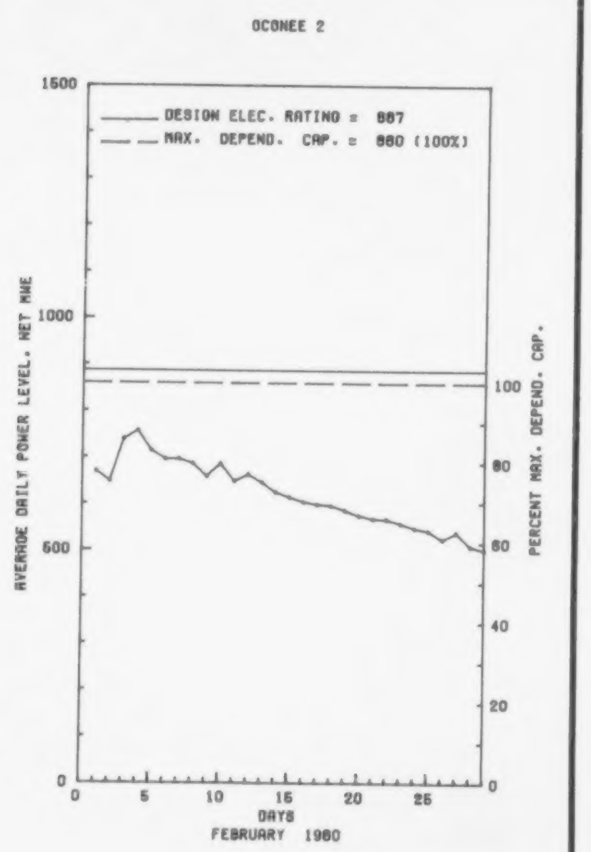
INSPECTION JANUARY 7-31 (80-04): THIS ROUTINE INSPECTION INVOLVED 19 INSPECTOR-HOURS ON SITE IN THE AREAS OF PLANT OPERATIONS, HOUSEKEEPING, FOLLOWUP OF IEB'S, IEC'S AND IE IN'S, PLANT TOURS AND WITNESSING A TEST. OF THE SIX AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN FIVE AREAS; ONE APPARENT DEVIATION WAS FOUND IN ONE AREA (INADEAUATE HOUSEKEEPING).

REVIEWED BY OMPA LOEB: DATE: 4/10/80

REVIEWED BY NRR DOR: DATE: [illegible]

OCONEE 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-270
2. Utility Contact: J. A. Reavis (704) 373-8552
3. Licensed Thermal Power (MWt): 2568
4. Nameplate Rating (Gross MWe): 1038 x 0.9 = 934
5. Design Electrical Rating (Net MWe): 887
6. Maximum Dependable Capacity (Gross MWe): 899
7. Maximum Dependable Capacity (Net MWe): 860
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

CAPACITY FACTORS CALCULATED WITH A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	696.0	1,440.0	48,001.0
12. Hours Reactor Was Critical	696.0	1,439.2	35,035.2
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	696.0	1,427.7	34,202.8
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,335,718	3,181,687	81,229,091
17. Gross Electrical Energy (MWH)	454,980	1,077,120	27,591,476
18. Net Electrical Energy (MWH)	431,069	1,025,965	26,189,723
19. Unit Service Factor	100.0	99.1	71.3
20. Unit Availability Factor	100.0	99.1	71.3
21. Unit Capacity Factor (MDC Net)	72.0	82.8	63.1
22. Unit Capacity Factor (DER Net)	69.8	80.3	61.5
23. Unit Forced Outage Rate	.0	.9	19.3

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUELING - MARCH 4, 1980 - 12 WEEKS
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
3	01/31/80	F	.0	H	5		RC	FUELXX	HOLD IN POWER INCREASE DUE TO XENON AND CORE FLUX IMBALANCE.
4	02/01/80	S	.0	H	5		RC	FUELXX	POWER REDUCTION DUE TO FUEL DEPLETION.
5	02/02/80	F	.0	H	5		RC	FUELXX	CORE FLUX IMBALANCE FORCED POWER REDUCTION TO STABILIZE.
6	02/03/80	S	.0	H	5		RC	FUELXX	POWER REDUCTION DUE TO FUEL DEPLETION.

SUMMARY

OPERATION WAS ROUTINE IN AN END-OF-CYCLE COASTDOWN.

LEGEND

[1] Type:
F-Forced
S-Scheduled

[2] Reason:
A-Equipment Failure
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error
H-Other

[3] Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Continuations
5-Load Reduction
9-Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source

OCONEE 3

FACILITY DATA

Facility Description

1. Location (state/county) (nearest population center)	South Carolina/Oconee 30 miles W of Greenville
2. Electric Reliability Council	Southeastern
3. Type of Reactor	PWR
4. Power Level (MWT)	2568
5. Design Electrical Rating (MWE Net)	887
6. Date of Initial Criticality	September 5, 1974
7. Date of Commercial Operation	December 16, 1974
8. Condenser Cooling Method	Once Through
9. Condenser Cooling Water Source	Keowee Lake

Utility & Contractor Information

10. Licensee	Duke Power Company
11. Corporate Address	422 S. Church Street Charlotte, North Carolina 28242
12. Corporate Contact	W. O. Parker, Jr.
13. Architect/Engineer	DPC/Bechtel
14. Nuclear Steam Supply System	B&W
15. Constructor	DPC

Regulatory Information

16. IE Region responsible	II
17. IE Branch Chief	R.C. Lewis
18. IE Senior Inspector	E.H. Verdery
19. IE Resident Inspector	F. Jape (803) 882-6927
20. Licensing Project Manager	G. Rivenbark
21. Docket Number	50-287
22. License Number/Date of Issue	DPR-55, 7/19/74
23. Public Document Room	Oconee County Library 201 S. Spring Street Walhalla, South Carolina 29691

INSPECTION STATUS

ENFORCEMENT STATUS

AS REQUIRED BY 10 CFR 20.203(F), EACH CONTAINER OF LICENSED MATERIAL SHALL BEAR A DURABLE, CLEARLY VISIBLE LABEL IDENTIFYING THE RADIOACTIVE CONTENTS AND PROVIDING SUFFICIENT INFORMATION TO PERMIT INDIVIDUALS HANDLING OR USING THE CONTAINERS, OR WORKING IN THE VICINITY THEREOF TO TAKE PRECAUTIONS TO AVOID OR MINIMIZE EXPOSURES. ON NOVEMBER 27, 1979, FIVE BAGS OF RADIOACTIVE MATERIAL LOCATED ON THE UNIT 3 LOW PRESSURE INJECTION DECK, WERE OBSERVED WITHOUT THE PROPER LABELS.
(7935 DEFICIENCY)

AS REQUIRED BY TECHNICAL SPECIFICATION 6.4, THE STATION SHALL BE OPERATED AND MAINTAINED IN ACCORDANCE WITH APPROVED PROCEDURES. (1) STATION PROCEDURE HP/0/B/1000/10, "WORKING LIMITS FOR CONTAMINATION CONTROL" STATES THAT IN AREAS WHERE CONTAMINATION EXISTS ABOVE THE WORKING LIMITS SPECIFIED, A POSTED AND/OR ROPED-OFF CONTAMINATION AREA WILL BE ESTABLISHED. PROCEDURE HP/0/B/1000/10 WAS NOT FOLLOWED IN THAT A POSTED AND/OR ROPED-OFF CONTAMINATION AREA WAS NOT ESTABLISHED ON NOVEMBER 19, 1979 AROUND FOUR UNIT 1 HIGH PRESSURE TURBINE HOLDDOWN BOLTS CONTAMINATED ABOVE THE LIMITS SPECIFIED BY PROCEDURE. (2) STATION PROCEDURE OP/0/A/1102/06, "REMOVAL AND RESTORATION OF STATION EQUIPMENT", ENCLOSURE 7.1, STEP 7.1.13 SPECIFIES THE PROPER VALVE OR BREAKER POSITION REQUIRED FOR RESTORATION OF A SYSTEM TO SERVICE. ON OCTOBER 16, 1979, WHILE RESTORING THE UNIT 3 LOW PRESSURE INJECTION SYSTEM "B" TRAIN TO SERVICE, VENT VALVE 3GWD-152 WAS NOT CLOSED AS CALLED FOR ON THE REMOVAL AND RESTORATION OF STATION EQUIPMENT CHECKLIST, ALTHOUGH THE CHECKLIST WAS INITIALLED AND DATED INDICATING THE VALVE WAS LEFT IN THE PROPER POSITION.
(7935 INFRACTION)

CONTRARY TO THE QA PLAN, ACCEPTED BY NRC JUNE 29, 1978 IN WHICH DPC COMMITTED TO RG 1.39, ON JANUARY 30 AND 31, 1980 COMMITMENTS TO ADHERE TO STATED HOUSEKEEPING PRACTICES WITHIN PLANT AREAS OCCUPIED BY SAFETY RELATED EQUIPMENT WERE NOT FOLLOWED. SPECIFICALLY THE LOW PRESSURE INJECTION, BUILDING SPRAY AND HIGH PRESSURE INJECTION ROOMS CONTAINED EXCESS MATERIALS, EQUIPMENT AND DEBRIS WHICH APPARENTLY REMAINED FROM PREVIOUS JOB ACTIVITIES IN THESE LOCATIONS.
(8004 DEVIATION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

+ DUE TO SHUT DOWN 3/15/80 FOR MODIFICATIONS REQUIRED BY NUREG 0578.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

100% POWER OPERATION.

+ LAST IE SITE INSPECTION DATE: FEBRUARY 25-29, 1980

+ INSPECTION REPORT NO: 50-287/80-04

SUMMARY

INSPECTION SUMMARY

+ INSPECTION NOVEMBER 11 AND 15-16 (79-37): THIS SPECIAL, ANNOUNCED INSPECTION INVOLVED 36 INSPECTOR-HOURS ON SITE IN THE AREAS OF TRANSIENT ANALYSIS AND REPORTABILITY REQUIREMENTS. OF THE TWO AREAS INSPECTED, TWO ITEMS OF NONCOMPLIANCE WERE FOUND IN ONE AREA (INFRACTION - INADEQUATE PROCEDURES TO CORRECT SPECIFIC AND FORESEEN POTENTIAL MALFUNCTIONS; AND INFRACTION - EXCESSIVE REACTOR COOLANT SYSTEM COOLDOWN RATE; ONE APPARENT DEVIATION WAS FOUND IN ONE AREA (FAILURE TO NOTIFY NRC WITHIN ONE HOUR WHEN UNIT 3 WAS IN AN UNEXPECTED OPERATING CONDITION).

INSPECTION JANUARY 21-25 AND JANUARY 28 - FEBRUARY 1 (80-03): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 18 INSPECTOR-HOURS ON SITE IN THE AREAS OF COORDINATION WITH OFFSITE SUPPORT AGENCIES, EMERGENCY FACILITIES, EQUIPMENT AND PROCEDURES, MEANS FOR DETERMINING A RELEASE, EMERGENCY TRAINING, EMERGENCY DRILLS, AND EMERGENCY ORGANIZATION. OF THE SIX AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION JANUARY 7-31 (80-04): THIS ROUTINE INSPECTION INVOLVED 19 INSPECTOR-HOURS ON SITE IN THE AREAS OF PLANT OPERATIONS, HOUSEKEEPING, FOLLOWUP OF IEB'S, IEC'S AND IE IN'S, PLANT TOURS AND WITNESSING A TEST. OF THE FIVE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN FOUR AREAS; ONE APPARENT DEVIATIONS WAS FOUND IN ONE AREA (INADEAUATE HOUSEKEEPING).

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

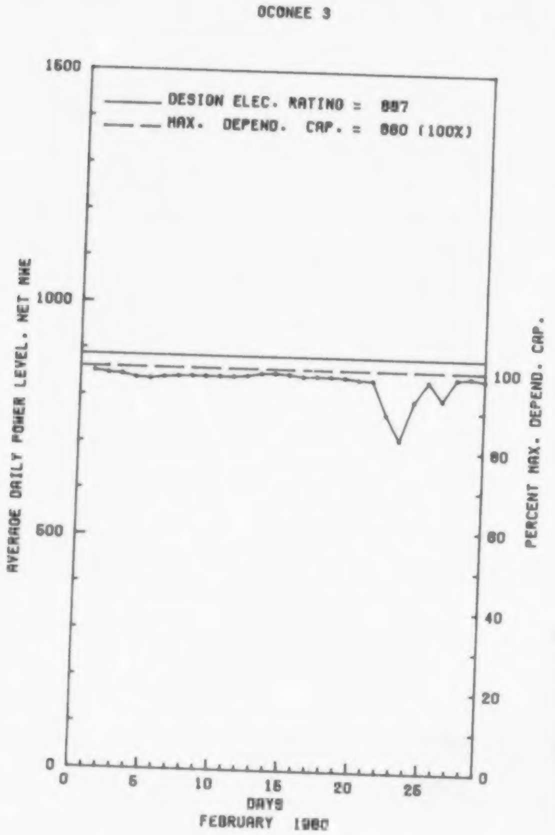
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
NONE				

REVIEWED BY OMPA LOEB: PEBobe **DATE:** 4/10/80

REVIEWED BY NRR DOR: **DATE:** 4/11/80

OCONEE 3

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-287
2. Utility Contact: J. A. Reavis (704) 373-8552
3. Licensed Thermal Power (MWt): 2568
4. Nameplate Rating (Gross MWe): 1038 x 0.9 = 934
5. Design Electrical Rating (Net MWe): 887
6. Maximum Dependable Capacity (Gross MWe): 899
7. Maximum Dependable Capacity (Net MWe): 860
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

CAPACITY FACTORS CALCULATED WITH A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	696.0	1,440.0	45,648.0
12. Hours Reactor Was Critical	696.0	1,440.0	33,333.9
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	696.0	1,440.0	32,501.9
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,760,327	3,656,329	78,037,934
17. Gross Electrical Energy (MWH)	605,630	1,260,900	27,012,164
18. Net Electrical Energy (MWH)	580,369	1,208,691	25,705,247
19. Unit Service Factor	100.0	100.0	71.2
20. Unit Availability Factor	100.0	100.0	71.2
21. Unit Capacity Factor (MDC Net)	97.0	97.6	65.2
22. Unit Capacity Factor (DER Net)	94.0	94.6	63.6
23. Unit Forced Outage Rate	.0	.0	17.4

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NRC MODIFICATIONS - MARCH 14, 1980 - 4 WEEKS
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
3	02/22/80	F	0	A	5		CH	PUMPXX	POWER REDUCED TO 85% FOR 3D1 HEATER DRAIN PUMP MAINTENANCE.
4	02/25/80	F	.0	A	5		HA	INSTRU	REDUCED POWER TO 75% TO CORRECT TURBINE CONTROL VALVE PROBLEM.

SUMMARY

OPERATION WAS AT NEAR FULL POWER DURING THE REPORT PERIOD, WITH A LOAD REDUCTION ON 02/22/80 FOR MAINTENANCE ON A HEATER DRAIN PUMP, AND ON 02/25/80 TO CORRECT A TURBINE CONTROL VALVE PROBLEM.

LEGEND

[1] Type:
F-Forced
S-Scheduled

[2] Reason:
A-Equipment Failure
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error
H-Other

[3] Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Continuations
5-Load Reduction
9-Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

OYSTER CREEK

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	New Jersey/Ocean 9 miles S of Toms River
2.	Electric Reliability Council	MAAC
3.	Type of Reactor	BWR
4.	Power Level (MWT)	1930
5.	Design Electrical Rating (MWE Net)	650
6.	Date of Initial Criticality	May 3, 1969
7.	Date of Commercial Operation	December 1969
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Barnegat Bay

Utility & Contractor Information

10.	Licensee	Jersey Central Power & Light Co.
11.	Corporate Address	Madison Avenue At Punch Bowl Road Morristown, New Jersey 07960
12.	Corporate Contact	I. Finfrock, Vice President Power Generation
13.	Architect/Engineer	Burns & Roe
14.	Nuclear Steam Supply System	GE
15.	Constructor	J. A. Jones

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	IE Senior Inspector	E. McCabe
19.	IE Principal Inspector	L. Briggs
20.	Licensing Project Manager	S. J. Nowicki
21.	Docket Number	50-219
22.	License Number/Date of Issue	DPR-16, 4/9/69
23.	Public Document Room	Ocean County Library 15 Hooper Avenue Toms River, New Jersey 08753

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO 10 CFR 20.201(B) ON MARCH 16, 1979, RADIATION SURVEYS NECESSARY TO ASSURE COMPLIANCE WITH 10 CFR 20.101, "EXPOSURE OF INDIVIDUALS TO RADIATION IN RESTRICTED AREAS" WERE NOT PERFORMED IN SUPPORT OF WORK ACTIVITIES CONDUCTED IN ACCORDANCE WITH RADIOACTIVE WORK PERMIT (RWP) NO. 062279. RWP NO. 062279 WAS ISSUED BASED ON SURVEY DATA COMPILED ON MARCH 1, 1979 AND INDICATED THAT THE RADIATION LEVELS IN THE WORK AREA WERE 1.0 MREM PER HOUR. ACTUAL RADIATION LEVELS IN THE WORK AREA MEASURED AS HIGH AS 200 MREM/HR ON MARCH 16, 1979. CONTRARY TO 10 CFR 20.201(B) BETWEEN MARCH 16 AND APRIL 26, 1979, NO EVALUATIONS WERE MADE TO DETERMINE THE RADIATION EXPOSURE RECEIVED AS A RESULT OF THE RADIATION DOSE TO THE SKIN OF THREE INDIVIDUALS WHO BECAME EXTENSIVELY CONTAMINATED WITH RADIOACTIVE WATER (PRIMARY COOLANT) WHILE PERFORMING ACTIVITIES IN ACCORDANCE WITH RADIOACTIVE WORK PERMIT NO. 062279. INITIAL RADIATION LEVELS AS HIGH AS 224 MRADS PER HOUR WERE MEASURED ON THE SKIN SURFACES OF THE INDIVIDUALS ON MARCH 16, 1979.
(7907 INFRACTION)

TECHNICAL SPECIFICATION 6.11, "RADIATION PROTECTION PROGRAMS," STATES, "PROCEDURES FOR PERSONNEL RADIATION PROTECTION SHALL BE PREPARED CONSISTENT WITH THE REQUIREMENTS OF 10 CFR PART 20 AND SHALL BE APPROVED, MAINTAINED AND ADHERED TO FOR ALL OPERATIONS INVOLVING PERSONNEL RADIATION EXPOSURE." CONTRARY TO THE ABOVE, ON MARCH 16, 1979, SIX PROCEDURES, WRITTEN PURSUANT TO THIS SPECIFICATION WERE NOT ADHERED TO IN REGARD TO ACTIVITIES CONDUCTED AS PERMITTED BY RWP NO. 062279, "CUT DWEDT DISCHARGE LINE AND CAP."
(7907 INFRACTION)

10 CFR 19.12, "INSTRUCTIONS TO WORKERS," STATES, "ALL INDIVIDUALS WORKING IN OR FREQUENTING ANY PORTION OF A RESTRICTED AREA SHALL BE KEPT INFORMED OF THE STORAGE, TRANSFER, OR USE OF RADIOACTIVE MATERIALS OR OF RADIATION IN SUCH PORTIONS OF THE RESTRICTED AREA; SHALL BE INSTRUCTED IN THE HEALTH PROTECTION PROBLEMS ASSOCIATED WITH EXPOSURE TO SUCH RADIOACTIVE MATERIALS OR RADIATION, IN PRECAUTIONS OR PROCEDURES TO MINIMIZE EXPOSURE, AND IN THE PURPOSES AND FUNCTIONS OF PROTECTIVE DEVICES EMPLOYED." THE EXTENT OF THOSE INSTRUCTIONS SHALL BE COMMENSURATE WITH POTENTIAL RADIOLOGICAL HEALTH PROTECTION PROBLEMS IN THE RESTRICTED AREA. IN ADDITION, HEALTH PHYSICS PROCEDURE 902.1, REVISION 13, RADIOACTIVE WORK PERMITS STATES IN SECTION 4.1, "WORKING PERSONNEL SHOULD BE BRIEFED ON THE WORK TO BE PERFORMED AND THE RADIOLOGICAL SAFETY PROCEDURES APPLICABLE TO THE JOB." CONTRARY TO THE ABOVE, ON MARCH 16, 1979, TWO INDIVIDUALS WERE DIRECTED TO PERFORM WORK IN ACCORDANCE WITH RADIOACTIVE WORK PERMIT NO. 062279, "CUT DWEDT DISCHARGE LINE AND CAP-SW CORNER OF RX BUILDING IN PIPE TUN" WITHOUT BEING INFORMED OF THE SPECIFIC RADIOLOGICAL HAZARDS ASSOCIATED WITH THIS ACTIVITY. AS A RESULT THE PERSONNEL WERE NOT AWARE OF THE POTENTIAL RADIOLOGICAL HEALTH PROTECTION PROBLEMS ASSOCIATED WITH THE WORK AND WERE NOT ADVISED OF PRECAUTIONS AND PROCEDURES TO MINIMIZE THEIR EXPOSURE.
(7907 INFRACTION)

CONTRARY TO 10 CFR 20.201(B) ON MARCH 16, 1979, RADIATION SURVEYS NECESSARY TO ASSURE COMPLIANCE WITH 10 CFR 20.101, "EXPOSURE OF INDIVIDUALS TO RADIATION IN RESTRICTED AREAS" WERE NOT PERFORMED IN SUPPORT OF WORK ACTIVITIES CONDUCTED IN ACCORDANCE WITH RADIOACTIVE WORK PERMIT (RWP) NO. 062279. RWP NO. 062279 WAS ISSUED BASED ON SURVEY DATA COMPILED ON MARCH 1, 1979 AND INDICATED THAT THE RADIATION LEVELS IN THE WORK AREA WERE 1.0 MREM PER HOUR. ACTUAL RADIATION LEVELS IN THE WORK AREA MEASURED AS HIGH AS 200 MREM/HR ON MARCH 16, 1979. CONTRARY TO 10 CFR 20.201(B) BETWEEN MARCH 16 AND APRIL 26, 1979, NO EVALUATIONS WERE MADE TO DETERMINE THE RADIATION EXPOSURE RECEIVED AS A RESULT OF THE RADIATION DOSE TO THE SKIN OF THREE INDIVIDUALS WHO BECAME EXTENSIVELY CONTAMINATED WITH RADIOACTIVE WATER (PRIMARY COOLANT) WHILE PERFORMING ACTIVITIES IN ACCORDANCE WITH RADIOACTIVE WORK PERMIT NO. 062279. INITIAL RADIATION LEVELS AS HIGH AS 224 MRADS PER HOUR WERE MEASURED ON THE SKIN SURFACES OF THE INDIVIDUALS ON MARCH 16, 1979.
(7907 INFRACTION)

(SEE CONTINUATION PAGE 0-9A)

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-043/ 03L	12/28/79	01/25/80	30 Day LER	Failure of one reactor building to torus vacuum breaker to open during surveillance testing
80-002/ 03L	01/16/80	02/15/80	30 Day LER	One fuel bundle found misoriented 180 degrees. Subsequent testing indicated no damage to the bundle
80-003/ 01T	01/16/80	01/30/80	24 Hours	Discovery of two crack indications in core spray sparge (system 2)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

- Three additional small cracks were identified in the core spray system II sparger during inspection and evaluation of previously identified and reported cracks.

PLANT STATUS:

- The plant is shutdown for refueling with return to operation scheduled for May 1, 1980.

- LAST IE SITE INSPECTION DATE: 3/15/80

- INSPECTION REPORT NO: 50-219/80-09

SUMMARY

INSPECTION SUMMARY

- 50-219/79-07 - Apr 18-20 & Jul 19-20: Special, unannounced inspection by 3 regional-based inspectors (64 hours) of the Radiation Protection Program as a result of written allegations. Areas inspected included Radiation Protection Procedures, Instructions to Personnel, Personnel Exposure Control and Surveys. Noncompliances: 5 (failure to perform surveys in accordance with 10 CFR 20.201 sufficient to assure compliance with 10 CFR 20.101; failure to perform survey in accordance with 10 CFR 20.201 sufficient to assure compliance with 10 CFR 20.103; failure to instruct workers in accordance with 10 CFR 19.12; failure to adhere to procedures in accordance with Technical Specification 6.11; failure to adhere to procedures in accordance with Technical Specification 6.8).

- 50-219/79-18 - Oct 1-5, 15-19, 29-31, Nov 1,2, & 6-8: Special, announced inspection by 6 IE Performance Appraisal Branch inspectors (752 hours) of the licensee's management controls over licensed activities. Noncompliances: 14 (failure to provide independent verification when removing/placing tags, jumpers, and leads; fire doors blocked open; inadequate drawing; inadequate procedures; failure to follow procedures; inadequate document control storage; failure to follow procedures; inadequate corrective action to QA findings; failure to audit; improper storage identification of weldrod; failure to establish calibration procedures; failure to survey effluent releases; failure to analyze Standby Gas Treatment System charcoal absorbers for removal efficiency; failure to report).

- 50-219/79-20 - Nov 27-30: Routine, unannounced inspection by one regional-based inspector (32 hours), initiated during a backshift, of: Security Plan, Security Program Audit, Security Organization, Physical Barriers (Protected Areas), Physical Barriers (Vital Area), Lighting, Access Control (Identification, Authorization, and Badging), Access Control (Escorting), Access Control (Search), Access Control (Vital Areas), Alarm Stations, Detection Aids, Assessment Aids, Communications, Testing and Maintenance, Compensatory Measures, Response, Locks, Keys, and Combinations, Records and Reports, followup of previously identified items and a review of licensee actions taken as a result of IE Bulletin 79-16. Noncompliances: None.

- 50-219/79-23 - Dec 9-14, 18-19 & Jan 4: Routine, unannounced inspection by one regional-based inspector (63 hours) of the Radioactive Waste Management System including: liquid and gaseous radioactive effluent releases; reactor coolant water quality; calibration and functional tests; standby gas treatment system; planning and preparation; solid radioactive waste, training, licensee action on IE Bulletin 79-19 and Licensee Event Report 50-219/79-41/3L and radiation safety control procedure adherence and practices. Noncompliances: 4 (failure to survey to meet 10 CFR 20.201; failure to meet 10 CFR 71.3; failure to maintain records pursuant to 10 CFR 71.62; failure to file request for change in Technical Specifications as required by 10 CFR 50.59(c)).

- 50-219/80-01 - Jan 4-6, 8-11 & 14: Routine, onsite, unannounced inspection by one regional-based inspector (59 hours) of 4 inspection areas: refueling preparation; refueling activities; independent inspection effort; and, IE Bulletin followup. Plant tours were also conducted. Noncompliances: None.

- 50-219/80-02 - Jan 15-18: Routine, onsite, announced inspection by one region-based inspector (34 hours) of five modifications, onsite LER followup; and independent effort. Noncompliances: None.

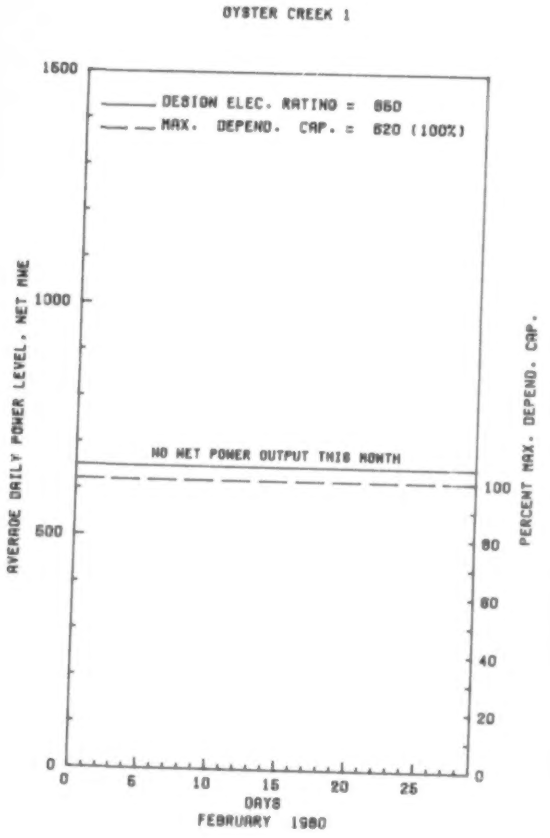
- 50-219/80-05 - Jan 29 - Feb 1: Routine, onsite, unannounced inspection by one regional-based inspector (42.5 hours) of 5 inspection areas: refueling activities; independent inspection effort; IE Bulletin followup; in-office licensee event report review; and, in-office periodic report review. Plant tours were also conducted. Noncompliances: None.

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 4-10-80

REVIEWED BY NRR DOR: ______ **DATE:** ______

OYSTER CREEK

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-219
2. Utility Contact: C. M. McClain (201) 455-8748
3. Licensed Thermal Power (MWt): 1930
4. Nameplate Rating (Gross MWe): 687.5 x 0.8 = 550
5. Design Electrical Rating (Net MWe): 650
6. Maximum Dependable Capacity (Gross MWe): 650
7. Maximum Dependable Capacity (Net MWe): 620
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

ITEM 21 CALCULATED BY USING A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	696.0	1,440.0	89,304.0
12. Hours Reactor Was Critical	.0	101.5	68,748.8
13. Reactor Reserve Shutdown Hours	.0	.0	468.2
14. Hours Generator On-Line	.0	98.4	67,403.9
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	157,611	114,036,769
17. Gross Electrical Energy (MWH)	0	53,670	38,935,985
18. Net Electrical Energy (MWH)	0	48,612	37,529,625
19. Unit Service Factor	.0	6.8	75.5
20. Unit Availability Factor	.0	6.8	75.5
21. Unit Capacity Factor (MDC Net)	.0	5.4	68.5
22. Unit Capacity Factor (DER Net)	.0	5.2	64.7
23. Unit Forced Outage Rate	.0	.0	6.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 05/01/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
1	01/05/80	S	696.0	C	4		ZZ	ZZZZZZ	REFUELING/MAINTENANCE OUTAGE CONTINUES.

SUMMARY

THE UNIT REMAINED SHUTDOWN THROUGHOUT THE REPORT PERIOD FOR A REFUELING AND MAINTENANCE OUTAGE.

LEGEND

[1] Type:
F Forced
S Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

ENFORCEMENT STATUS

CONTRARY TO 10 CFR 20.103(A)(3) ON MARCH 16, 1979, SURVEYS OF CONCENTRATIONS OF RADIOACTIVE MATERIALS IN AIR WERE NOT MADE TO SUPPORT WORK WHICH HAD THE POTENTIAL FOR SUBJECTING PERSONNEL TO EXPOSURE FROM AIRBORNE RADIOACTIVE MATERIALS IN AIR. SUCH WORK WAS PERFORMED BY INDIVIDUALS IN ACCORDANCE WITH THE FOLLOWING RADIOACTIVE WORK PERMITS (RWPS): 061979 - RUN TEMP PIPE LINE, DWEDT LINE-HUB DRAIN IN CSHX AREA; 062279 - CUT DWEDT DISCHARGE LINE AND CAP-SW CORNER OF RX BUILDING IN PIPE TUN. CONTRARY TO 10 CFR 20.103(A)(3) FOR THE PERIOD BETWEEN MARCH 16 AND APRIL 20, 1979, NO EVALUATIONS SUCH AS MEASUREMENTS OF RADIOACTIVITY IN THE BODY, MEASUREMENTS OF RADIOACTIVITY EXCRETED FROM THE BODY OR ANY COMBINATION OF SUCH MEASUREMENTS, WAS MADE ON THREE INDIVIDUALS PERFORMING WORK IN ACCORDANCE WITH RADIOACTIVE WORK PERMIT 062279, IN WHICH THE PERSONNEL WERE SUBJECT TO EXTENSIVE CONTAMINATION RESULTING IN A DOSE RATE AS HIGH AS 224 MRADS PER HOUR ON THE UPPER PORTIONS OF THE PERSONS HEAD AND FACE.
(7907 INFRACTION)

TECHNICAL SPECIFICATION 6.8.1, "PROCEDURES" STATES, "WRITTEN PROCEDURES SHALL BE ESTABLISHED, IMPLEMENTED (ADHERED TO), AND MAINTAINED THAT MEET OR EXCEED THE REQUIREMENTS OF SECTION 5.1 AND 5.3 OF AMERICAN NATIONAL STANDARD N18.7-1972 AND APPENDIX 'A' OF THE NUCLEAR REGULATORY COMMISSION'S REGULATORY GUIDE 1.33-1972...." PROCEDURE 79-7, REVISION 0, REPAIR OF DWEDT DISCHARGE LINE, WRITTEN IN ACCORDANCE WITH THIS SPECIFICATION, STATES THE FOLLOWING: "3.0 PREREQUISITES...3.5. ENSURE THE DISCHARGE LINE IS APPROPRIATELY ISOLATED, DRAINED AND TAGGED PRIOR TO CUTTING INTO IT. 5.0 INSTRUCTIONS...5.1.4. ENSURE THE SECTION OF THE DISCHARGE LINE TO BE CUT IS ISOLATED, DRAINED AND TAGGED. CONTACT THE GROUP SHIFT SUPERVISOR FOR NOTIFICATION." CONTRARY TO THE ABOVE, ON MARCH 16, 1979, PROCEDURE 79-7, REVISION 0, WAS NOT ADHERED TO IN THAT THE DISCHARGE LINE WAS NOT ISOLATED AND DRAINED PRIOR TO CUTTING THE PIPE. CONSEQUENTLY, SEVERAL PERSONS RECEIVED EXTENSIVE RADIOACTIVE CONTAMINATION WHEN THEY WERE SPRAYED WITH THE LIQUID CONTENTS (PRIMARY REACTOR COOLANT) OF THE PIPE WHICH WAS UNDER AN ESTIMATED 22 PSI OF PRESSURE WHEN THE PIPE CUT WAS MADE. RADIATION LEVELS AS HIGH AS 224 MRAD/HR WERE MEASURED OVER THE ENTIRE SKIN SURFACES OF ONE OF THE INDIVIDUALS AND IN VARIOUS LOCATIONS OF THE BODY OF THE TWO OTHER INDIVIDUALS WHO WERE EXPOSED.
(7907 INFRACTION)

TECHNICAL SPECIFICATION 6.11, "RADIATION PROTECTION PROGRAMS," STATES, "PROCEDURES FOR PERSONNEL RADIATION PROTECTION SHALL BE PREPARED CONSISTENT WITH THE REQUIREMENTS OF 10 CFR PART 20 AND SHALL BE APPROVED, MAINTAINED AND ADHERED TO FOR ALL OPERATIONS INVOLVING PERSONNEL RADIATION EXPOSURE." CONTRARY TO THE ABOVE, ON MARCH 16, 1979, SIX PROCEDURES, WRITTEN PURSUANT TO THIS SPECIFICATION WERE NOT ADHERED TO IN REGARD TO ACTIVITIES CONDUCTED AS PERMITTED BY RWP NO. 062279, "CUT DWEDT DISCHARGE LINE AND CAP."
(7907 INFRACTION)

10 CFR 19.12, "INSTRUCTIONS TO WORKERS," STATES, "ALL INDIVIDUALS WORKING IN OR FREQUENTING ANY PORTION OF A RESTRICTED AREA SHALL BE KEPT INFORMED OF THE STORAGE, TRANSFER, OR USE OF RADIOACTIVE MATERIALS OR OF RADIATION IN SUCH PORTIONS OF THE RESTRICTED AREA; SHALL BE INSTRUCTED IN THE HEALTH PROTECTION PROBLEMS ASSOCIATED WITH EXPOSURE TO SUCH RADIOACTIVE MATERIALS OR RADIATION, IN PRECAUTIONS OR PROCEDURES TO MINIMIZE EXPOSURE, AND IN THE PURPOSES AND FUNCTIONS OF PROTECTIVE DEVICES EMPLOYED." THE EXTENT OF THOSE INSTRUCTIONS SHALL BE COMMENSURATE WITH POTENTIAL RADIOLOGICAL HEALTH PROTECTION PROBLEMS IN THE RESTRICTED AREA. IN ADDITION, HEALTH PHYSICS PROCEDURE 902.1, REVISION 13, RADIOACTIVE WORK PERMITS STATES IN SECTION 4.1, "WORKING PERSONNEL SHOULD BE BRIEFED ON THE WORK TO BE PERFORMED AND THE RADIOLOGICAL SAFETY PROCEDURES APPLICABLE TO THE JOB." CONTRARY TO THE ABOVE, ON MARCH 16, 1979, TWO INDIVIDUALS WERE DIRECTED TO PERFORM WORK IN ACCORDANCE WITH RADIOACTIVE WORK PERMIT NO. 062279, "CUT DWEDT DISCHARGE LINE AND CAP-SW CORNER OF RX BUILDING IN PIPE TUN" WITHOUT BEING INFORMED OF THE SPECIFIC RADIOLOGICAL HAZARDS ASSOCIATED WITH THIS ACTIVITY. AS A RESULT THE PERSONNEL WERE NOT AWARE OF THE POTENTIAL RADIOLOGICAL HEALTH PROTECTION PROBLEMS ASSOCIATED WITH THE WORK AND WERE NOT ADVISED OF PRECAUTIONS AND PROCEDURES TO MINIMIZE THEIR EXPOSURE.
(7907 INFRACTION)

LICENSEE EMERGENCY PROCEDURES USED TO COPE WITH MAY 2, 1979 EVENT WERE NOT APPROPRIATE FOR THE CIRCUMSTANCES AND WERE A MAJOR CAUSATIVE FACTOR IN THAT EVENT. 10 CFR 50 APPENDIX B, CRITERION 5.
(7910 VIOLATION)

10 CFR 20.201, "SURVEYS" STATES IN PARAGRAPH (B) THAT "EACH LICENSEE SHALL MAKE OR CAUSE TO BE MADE SUCH SURVEYS AS MAY BE NECESSARY FOR HIM TO COMPLY WITH THE REGULATIONS IN THIS PART." PARAGRAPH (A) OF 20.201 DEFINES "SURVEY" AS "AN EVALUATION OF THE RADIATION HAZARDS INCIDENT TO THE PRODUCTION, USE, RELEASE, DISPOSAL OR PRESENCE OF RADIOACTIVE MATERIALS...." 10 CFR 20.301, "WASTE DISPOSAL," GENERAL REQUIREMENTS, SPECIFIES THAT NO LICENSEE SHALL DISPOSE OF LICENSED MATERIAL EXCEPT BY TRANSFER TO AN AUTHORIZED RECIPIENT AS PROVIDED IN 10 CFR 30... 10 CFR 30.41, "TRANSFER OF BYPRODUCT MATERIAL" REQUIRES THAT NO LICENSE SHALL TRANSFER BYPRODUCT MATERIAL EXCEPT AS AUTHORIZED PURSUANT TO THIS SECTION. THIS SECTION PERMITS TRANSFERS TO PERSONS AUTHORIZED TO RECEIVE BYPRODUCT UNDER TERMS OF A LICENSE ISSUED BY AN AGREEMENT STATE. CHEM NUCLEAR SYSTEMS, INC.'S AGREEMENT STATE LICENSE FOR THE BARNWELL, SOUTH CAROLINA BURIAL SITE (SOUTH CAROLINA RADIOACTIVE MATERIAL LICENSE NO. 097) SPECIFIES IN LICENSE CONDITION 25 THAT SOLIDIFIED RADIOACTIVE WASTE HAVING DETECTABLE FREE STANDING LIQUIDS WILL NOT BE PERMITTED TO BE RECEIVED BY THE BURIAL SITE. SOUTH CAROLINA HAS DEFINED "NO DETECTABLE FREE STANDING LIQUID" AS LESS THAN 1.0% LIQUID BY VOLUME. CONTRARY TO THE ABOVE, NO SURVEYS WERE PERFORMED TO DETERMINE THE AMOUNT, IF ANY, OF FREE STANDING LIQUID IN A SHIPMENT OF DEWATERED RESINS (SHIPMENT NO. OC-1148-79) SHIPPED BY THE LICENSEE TO THE BURIAL SITE ON DECEMBER 10, 1979. NOTWITHSTANDING THE ABOVE SHIPMENT, NO DOCUMENTATION WAS AVAILABLE TO INDICATE PREVIOUS SHIPMENTS OF SOLIDIFIED WASTE HAD BEEN SURVEYED WITH RESPECT TO THE ABOVE.
(7923 INFRACTION)

10 CFR 50, DOMESTIC LICENSING OF PRODUCTION AND UTILIZATION FACILITIES, SECTION 50.59, CHANGES, TESTS AND EXPERIMENTS, SPECIFIES IN PARAGRAPH (A) THAT THE HOLDER OF A LICENSE AUTHORIZING OPERATION OF A UTILIZATION FACILITY MAY MAKE CHANGES IN THE FACILITY AS DESCRIBED IN THE SAFETY ANALYSIS REPORT, WITHOUT PRIOR COMMISSION APPROVAL, UNLESS THE PROPOSED CHANGE INVOLVES A CHANGE IN THE TECHNICAL SPECIFICATIONS INCORPORATED IN THE LICENSE OR AN UNREVIEWED SAFETY QUESTION. ADDITIONALLY, 10 CFR 50.59(C) SPECIFIES THAT THE HOLDER OF A LICENSE AUTHORIZING OPERATION OF A UTILIZATION FACILITY WHO DESIRES TO MAKE A CHANGE IN TECHNICAL SPECIFICATION OR A CHANGE IN THE FACILITY AS DESCRIBED IN THE SAFETY ANALYSIS REPORT SHALL SUBMIT ON APPLICATION FOR AMENDMENT OF HIS LICENSE. CONTRARY TO THE ABOVE, AS OF DECEMBER 10, 1979, NO APPLICATION FOR AMENDMENT TO THE FACILITY LICENSE HAD BEEN MADE WHICH ADDRESSED CHANGES TO THE FACILITY TECHNICAL SPECIFICATION IN ORDER TO INCLUDE RADIOACTIVE EFFLUENT RELEASES FROM THE NEW RADWASTE BUILDING. THIS BUILDING, CONSTRUCTED AND PLACED INTO OPERATION BY THE LICENSEE CONSTITUTED A CHANGE IN THE FACILITY AS DESCRIBED IN THE SAFETY ANALYSIS REPORT. THE LICENSEE HAD SUBMITTED A SAFETY ANALYSIS REPORT PURSUANT TO 10 CFR 50.59(B) AND PROPOSED TECHNICAL SPECIFICATIONS FOR BUILDING SETTLEMENT IN SOIL. HOWEVER, SINCE INITIAL INTORDUCTION OF RADIOACTIVE MATERIAL INTO THIS BUILDING IN OCTOBER 1978 NO SUBMITTAL HAS BEEN MADE ADDRESSING THE ABOVE RELEASES.
(7923 INFRACTION)

10 CFR 71.3, "REQUIREMENT FOR LICENSE," REQUIRES THAT NO LICENSEE SUBJECT TO THE REGULATIONS IN THIS PART SHALL (A) DELIVER ANY LICENSED MATERIALS TO A CARRIER FOR TRANSPORT OR (B) TRANSPORT LICENSED MATERIAL EXCEPT AS AUTHORIZED IN A GENERAL LICENSE OR SPECIFIC LICENSE ISSUED BY THE COMMISSION, OR AS EXEMPTED IN THIS PART. CONTRARY TO THE ABOVE, ON DECEMBER 10, 1979, AND AT OTHER TIMES, SHIPMENTS OF LICENSED MATERIALS IN EXCESS OF A TYPE A QUANTITY WERE DELIVERED TO A CARRIER FOR TRANSPORT WITHOUT A GENERAL OR SPECIFIC LICENSE AND NO EXEMPTION IN 10 CFR 71 WAS APPLICABLE.
(7923 INFRACTION)

10 CFR 71.62, "RECORDS," REQUIRES IN PART THAT THE LICENSEE SHALL MAINTAIN FOR A PERIOD OF 2 YEARS AFTER ITS GENERATION, A RECORD OF EACH SHIPMENT OF MORE THAN A TYPE A QUANTITY OF RADIOACTIVE MATERIAL AS DEFINED IN 71.4(G), IN A SINGLE PACKAGE, SHOWING WHERE APPLICABLE, THE RESULTS OF THE DETERMINATIONS REQUIRED BY 71.54, "ROUTINE DETERMINATIONS." THESE DETERMINATIONS INCLUDE DETERMINATIONS THAT: (A) THE PACKAGING HAS NOT BEEN SIGNIFICANTLY DAMAGED...(C) THE CLOSURE OF THE PACKAGE AND ANY SEALING GASKETS ARE PRESENT AND ARE FREE FROM DEFECTS...AND (I) THE PACKAGE HAS BEEN LOADED AND CLOSED IN ACCORDANCE WITH WRITTEN PROCEDURES. ADDITIONALLY, 71.62 REQUIRES RECORDS TO BE MAINTAINED OF THE RESULTS OF DETERMINATION MADE PURSUANT TO 10 CFR 71.53 INCLUDING: DETERMINATIONS THAT THERE ARE NO CRACKS, VOIDS OR OTHER DEFECTS WHICH COULD REDUCE THE EFFECTIVENESS OF THE PACKAGING PRIOR TO ITS FIRST USE. CONTRARY TO THE ABOVE, THE FOLLOWING SHIPMENTS CONTAINED GREATER THAN A TYPE A QUANTITY OF RADIOACTIVE MATERIAL IN A SINGLE PACKAGE AND WERE SHIPPED FROM THE LICENSEE'S FACILITY WITHOUT RECORD OF THE ABOVE DETERMINATIONS BEING PERFORMED: 05 1018, 1025, 1009, 1073 (1979).
(7923 DEFICIENCY)

NON FLAME RETARDENT WOODEN CRATES STORED ON 119' LEVEL OF THE REACTOR BUILDING CONTRARY TO SECTION 5.1.12 OF PROCEDURE 120, FIRE HAZARDS, REVISION 5.
(7924 INFRACTION)

PALISADES

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Michigan/Van Buren 5 miles S of South Haven
2.	Electric Reliability Council	ECAR
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2530
5.	Design Electrical Rating (MWE Net)	805
6.	Date of Initial Criticality	May 24, 1971
7.	Date of Commercial Operation	December 31, 1971
8.	Condenser Cooling Method	Forced Draft Cooling Towers
9.	Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10.	Licensee	Consumers Power Company
11.	Corporate Address	212 W. Michigan Avenue Jackson, Michigan 49201
12.	Corporate Contact	R. B. DeWitt
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Combustion Engineering
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Senior Inspector	D.C. Boyd
19.	IE Resident Inspector	B. Jorgenson (616) 764-8971
20.	Licensing Project Manager	R. Silver
21.	Docket Number	50-255
22.	License Number/Date of Issue	DPR-20, 3/24/71
23.	Public Document Room	Kalamazoo Public Library 315 South Rose Street Reference Department Kalamazoo, Michigan 49006

INSPECTION STATUS

ENFORCEMENT STATUS

10 CFR 20.101(A) PLACES A LIMIT OF 1.25 REM PER CALENDAR QUARTER ON AN INDIVIDUAL'S OCCUPATIONAL DOSE TO THE WHOLE BODY. THE EXCEPTION PROVIDED FOR BY 10 CFR 20.101(B), WHICH PERMITS QUARTERLY WHOLE BODY DOSE FROM OCCUPATIONAL EXPOSURE UP TO 3 REM IF THE ACCUMULATED WHOLE BODY DOSE DOES NOT EXCEED 5 (N-18), DOES NOT APPLY TO EIGHTEEN-YEAR-OLD INDIVIDUALS, AS "N" IS DEFINED AS THE INDIVIDUAL'S AGE IN YEARS AT HIS LAST BIRTHDAY. CONTRARY TO THE ABOVE, THE LICENSEE REPORTED BY LETTER DATED OCTOBER 26, 1979, THAT AN EIGHTEEN-YEAR-OLD WORKER HAD RECEIVED 1.7 REMS DURING THE THIRD CALENDAR QUARTER OF 1979. THIS WHOLE BODY DOSE EXCEEDS THE APPLICABLE 1.25 REM LIMIT.
(7919 INFRACTION)

10 CFR 20.201(B) REQUIRES EACH LICENSEE TO MAKE OR CAUSE TO BE MADE SUCH SURVEYS AS MAY BE NECESSARY FOR HIM TO COMPLY WITH THE REGULATION IN THIS PART. 10 CFR 20.201(A) DEFINES A SURVEY AS USED IN THE REGULATIONS TO MEAN AN EVALUATION OF THE RADIATION HAZARDS INCIDENT TO THE PRODUCTION, USE, RELEASE, DISPOSAL, OR PRESENCE OF RADIOACTIVE MATERIALS OR OTHER SOURCES OF RADIATION UNDER A SPECIFIC SET OF CONDITIONS. WHEN APPROPRIATE, SUCH EVALUATION INCLUDES A PHYSICAL SURVEY OF THE LOCATION OF MATERIALS AND EQUIPMENT, AND MEASUREMENTS OF LEVELS OF RADIATION OR CONCENTRATIONS OF RADIOACTIVE MATERIAL PRESENT. CONTRARY TO THE ABOVE, THE LICENSEE FAILED TO MAKE A PROPER EVALUATION FOR THE PRESENCE OF RADIOACTIVE MATERIAL IN PLANT REFUSE BEING RELEASED TO A LANDFILL FOR BURIAL IN 1978 AND POSSIBLY IN EARLY 1979.
(7919 INFRACTION)

TECHNICAL SPECIFICATIONS 6.8.1 AND 6.11.1 REQUIRE THAT RADIATION PROTECTION PROCEDURES BE IMPLEMENTED AND ADHERED TO. PROCEDURE H.P. 8.4, "WHOLE BODY COUNTING PROGRAM," IN SECTION 5.2.1, "EXIT WHOLE BODY COUNTING," STATES, "ALL PERSONNEL, WHETHER PERMANENT OR TEMPORARY, WILL BE WHOLE BODY COUNTED AT THE TIME OF THEIR FILM BADGE TERMINATION UNLESS THE PLANT HEALTH PHYSICIST DEEMS A COUNT NOT NECESSARY BASED ON THE WORK PERFORMED AND THE AREA IT WAS PERFORMED IN BY THE INDIVIDUAL." CONTRARY TO THE ABOVE, A WORKER WHO HAD WORKED IN CONTAINMENT AND THE STEAM GENERATOR DURING THE EARLY PART OF THE CURRENT OUTAGE WAS NOT GIVEN A WHOLE BODY COUNT UPON HIS TERMINATION FOLLOWING WORK ON OCTOBER 8, 1979.
(7919 INFRACTION)

ENVIRONMENTAL TECHNICAL SPECIFICATION 3.9.8 REQUIRES THAT ALL DISCHARGES FROM THE PLANT INTO LAKE MICHIGAN BE LIMITED TO A RANGE OF 6.5 TO 9.5 PH UNITS. CONTRARY TO THE ABOVE, ON MARCH 21, 1979, WATER WAS DISCHARGED TO THE LAKE WITH A CALCULATED PH OF 5.2.
(7921 INFRACTION)

ENVIRONMENTAL TECHNICAL SPECIFICATION 3.9.9 REQUIRES THAT THERMAL DISCHARGES OF THE TOTAL BLOWDOWN FROM THE CLOSED-CYCLE COOLING SYSTEM MIXED WITH DILUTION WATER SHALL BE LIMITED TO AN INCREASE OF NO MORE THAN 5F DEGREES ABOVE THE AMBIENT TEMPERATURE OF THE RECEIVING WATER AT THE SHORELINE OF THE LAKE AND THAT HEAT REJECTION TO THE LAKE FROM THE DILUTED TOTAL BLOWDOWN SHALL NOT EXCEED 0.5 BILLION BRITISH THERMAL UNITS (BTU'S) PER HOUR. IF THESE LIMITS ARE EXCEEDED BY MORE THAN 20% FOR MORE THAN 12 HOURS, THE LICENSEE SHALL TAKE ACTION TO DETERMINE THE CAUSE AND MODIFY PLANT OPERATIONS TO RESOLVE THE PROBLEM. IF THESE LIMITS ARE EXCEEDED FOR ANY PERIOD OF MORE THAN 24 HOURS (AND UP TO 7 DAYS ON LOSS OF ONE DILUTION PUMP), THE LICENSEE SHALL REDUCE POWER ACCORDINGLY TO RESTORE COMPLIANCE WITH SPECIFICATIONS TO PROTECT THE AQUATIC BIOTA. CONTRARY TO THE ABOVE, ON DECEMBER 19, 1978, THERMAL DISCHARGES TO LAKE MICHIGAN GREATER THAN 6F DEGREES ABOVE THE AMBIENT TEMPERATURE EXISTED FOR A PERIOD OF MORE THAN 24 HOURS.
(7921 INFRACTION)

THE LICENSEE DID NOT CONDUCT PROPER PERSONNEL SCREENING IN ACCORDANCE WITH SECURITY PLAN COMMITMENTS.
(7923 INFRACTION)

THE LICENSEE FAILED TO PROVIDE POSITIVE CONTROL OF ACCESS TO THE PROTECTED AREA AT ONE ACCESS PORTAL.
(7923 DEFICIENCY)

PART OF PHYSICAL BARRIER TO PROTECTED AREA DID NOT OFFER PENETRATION RESISTANCE EQUAL TO THAT OF THE FENCE.
(7923 DEFICIENCY)

THE LICENSEE DOES NOT CONDUCT ADEQUATE SEARCHES IN ACCORDANCE WITH A SECURITY PROCEDURE COMMITMENT.
(7923 DEFICIENCY)

THE LICENSEE FAILED TO ADEQUATELY PROTECT ONE CENTRAL COMMUNICATION SYSTEM.
(7923 DEFICIENCY)

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
		NONE		

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENTS

- The plant commenced a scheduled 10-week refueling outage on September 8, 1979. (79-15)
- Cracking was discovered in feedlines to both steam generators during examinations pursuant to IE Bulletin 79-13. The licensee decided to replace an approximate 28-foot horizontal section to each generator during the refueling outage. (79-15)

PLANT STATUS

Reactor shutdown on September 10, 1979, to begin a 10-week scheduled refueling and maintenance outage. Start-up tentatively scheduled for late April 1980

- LAST IE SITE INSPECTION DATE: December 10-14, 1979
- INSPECTION REPORT NO: 79-23

SUMMARY

INSPECTION SUMMARY

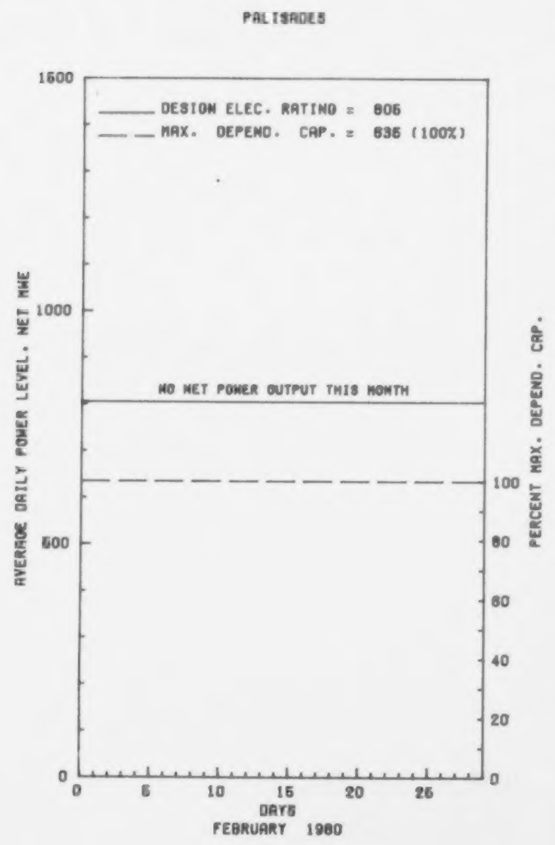
- Inspection during September 1979, (79-15): Routine inspection by the assigned NRC Resident Inspector covering reportable events, IE Bulletins, Maintenance Department activities, procedures, and cleanliness. The inspection involved 95 inspector hours onsite. Of the five areas inspected, no items of noncompliance or deviations were identified in four areas. Three noncompliance items were identified in the remaining area (containment integrity violated; failure to adhere to required procedure; and failure to confirm condition of manual containment isolation valves after refueling outages).
- Inspection on November 8-9 and 14-16, (79-19): Routine, unannounced inspection of radiation protection program associated with the refueling outage, including: radiation protection procedures; advance planning and preparation; training; external exposure control; internal exposure control; ALARA; posting and control; radioactive and contaminated material control; surveys; IE Bulletin 78-08; IE Information Notice 79-08; audits; instruments and equipment; secondary system demineralizer resin; and IE Bulletin 79-19. The inspection involved 47.5 inspector hours onsite by one NRC inspector. Of the 15 areas inspected, one apparent item of noncompliance identified in the area of external exposure control involved the overexposure of an eighteen-year-old; one apparent item of noncompliance identified in the area of radiation protection procedures involved a failure to follow procedures; and one apparent item of noncompliance identified in the area of the secondary systems demineralizer resin involved failure to make proper evaluations for presence of radioactive material in plant refuse before release for burial.
- Inspection on December 10-14, (79-23): Included a review of Security Plan; Security Program Audit; Security Organization; Protected Area Physical Barriers; Vital Area Physical Barriers; Lighting; Access Control (Identification, Authorization, and Badging); Access Control (Search); Access Control (Escorting); Access Control (Vital Areas); Alarm Stations; Detection Aids; Assessment Aids; Communications; Testing and Maintenance; Compensatory Measures; Power Supply; Response; Locks, Keys, and Combinations; Records and Reports; Followup on Previous Item of Noncompliance/Deviations; Followup on IE Bulletin No. 79-16, titled "Vital Area Access Controls." The inspection involved 93 inspector hours onsite by two NRC inspectors. The inspection began during the day shift period; 22 inspection hours were accomplished during offshift periods. The licensee was found to be in compliance with NRC requirements within the 20 areas examined during this inspection except for the following 5 items: Access Control (Identification, Authorization and Badging) - The licensee did not conduct proper personnel screening in accordance with security plan commitments. This is considered a repeat item of noncompliance; Access Control (Identification, Authorization and Badging) - The licensee failed to provide positive control of access to the protected area at one access portal; Access Control (Search) - The licensee does not conduct adequate searches in accordance with a security procedural commitment; Communications - The licensee failed to adequately protect one central communication system; and Physical Barriers (Protected Area) - A part of the perimeter barrier did not offer penetration resistance equal to that of the fence. The items of noncompliance from the previous security inspection 79-10, are closed with the exception of those concerning: (1) Background screening of personnel, (audit in progress) and (2) Tamper indication on the intrusion alarm system (submitted to NMSS for resolution).

REVIEWED BY OMPA LOEB: Dmk Beebe **DATE:** 04/10/80

REVIEWED BY NRR DOR: ____________ **DATE:** ____________

PALISADES

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-255
2. Utility Contact: P. J. Bodtke (616) 764-8913
3. Licensed Thermal Power (MWt): 2530
4. Nameplate Rating (Gross MWe): 955 x 0.85 = 812
5. Design Electrical Rating (Net MWe): 805
6. Maximum Dependable Capacity (Gross MWe): 675
7. Maximum Dependable Capacity (Net MWe): 635
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	696.0	1,440.0	71,871.0
12. Hours Reactor Was Critical	.0	.0	39,564.8
13. Reactor Reserve Shutdown Hours	.0	.0	.3
14. Hours Generator On-Line	.0	.0	37,426.5
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	0	71,511,480
17. Gross Electrical Energy (MWH)	0	0	22,032,230
18. Net Electrical Energy (MWH)	0	0	20,670,775
19. Unit Service Factor	.0	.0	52.1
20. Unit Availability Factor	.0	.0	52.1
21. Unit Capacity Factor (MDC Net)	.0	.0	45.3
22. Unit Capacity Factor (DER Net)	.0	.0	35.7
23. Unit Forced Outage Rate	.0	.0	36.2

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 04/25/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
13	09/08/79	S	696.0	D	4				TMI MODIFICATIONS AND ANCHOR BOLT INSPECTIONS/REPAIRS ACCORDING TO IE BULLETIN 79-02.

SUMMARY

THE UNIT REMAINED SHUTDOWN THROUGHOUT THE REPORT PERIOD FOR SEISMIC PIPE HANGER/ANCHOR BOLT INSPECTIONS PER IE BULLETIN 79-02, AND FOR TMI-RELATED MODIFICATIONS.

LEGEND

[1] Type:
- F-Forced
- S-Scheduled

[2] Reason:
- A-Equipment Failure
- B-Maintenance or Test
- C-Refueling
- D-Regulatory Restriction
- E-Operator Training & License Examination
- F-Administrative
- G-Operational Error
- H-Other

[3] Method:
- 1-Manual
- 2-Manual Scram
- 3-Automatic Scram
- 4-Continuations
- 5-Load Reduction
- 9-Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

PEACH BOTTOM 2

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Pennsylvania/York 19 miles S of Lancaster
2.	Electric Reliability Council	MAAC
3.	Type of Reactor	BWR
4.	Power Level (MWT)	3293
5.	Design Electrical Rating (MWE Net)	1065
6.	Date of Initial Criticality	September 16, 1973
7.	Date of Commercial Operation	July 5, 1974
8.	Condenser Cooling Method	Once Through with Cooling Towers
9.	Condenser Cooling Water Source	Conowingo Pond/Susquehanna River

Utility & Contractor Information

10.	Licensee	Philadelphia Electric Company
11.	Corporate Address	2301 Market Street Philadelphia, Pennsylvania 19101
12.	Corporate Contact	S. L. Daltroff, Vice President Electric Production
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	GE
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	IE Senior Inspector	E. McCabe
19.	IE Resident Inspector	E. Greenman 717-456-7614
20.	Licensing Project Manager	Dick Clark
21.	Docket Number	50-277
22.	License Number/Date of Issue	DPR-44, 8/8/73
23.	Public Document Room	Government Publications Section State Library of Pennsylvania Education Building Commonwealth and Walnut Street Harrisburg, Pennsylvania 17126

INSPECTION STATUS

ENFORCEMENT STATUS

TECHNICAL SPECIFICATION 3.14.B.3, "CO2 FIRE PROTECTION SYSTEMS" STATES, "THE DIESEL GENERATOR CO2 FIRE PROTECTION SHALL BE OPERABLE WHEN THE DIESEL GENERATORS ARE REQUIRED TO BE OPERABLE WITH THE SYSTEM COMPRISED OF: ...B. AN OPERABLE FLOW PATH TO EACH ROOM." TECHNICAL SPECIFICATION 3.14.13.4 STATES IN PART, "IF THE REQUIREMENTS OF 3.14.B.1, 2, OR 3 CANNOT BE MET, (A.) ESTABLISH A CONTINUOUS FIRE WATCH...FOR THE UNPROTECTED AREA (HPCI, CABLE SPREADING, COMPUTER, DIESEL GENERATOR) WITHIN ONE HOUR." CONTRARY TO THE ABOVE, ON AUGUST 21, 1979, WITH UNITS 2 AND 3 OPERATING IN THE RUN MODE AND DIESEL GENERATOR OPERABILITY REQUIRED PURSUANT TO TECHNICAL SPECIFICATIONS 3.9.A AND 3.9.B, AFTER PROPERLY ESTABLISHING A REQUIRED FIRE WATCH WHILE A PORTION OF THE DIESEL GENERATOR BUILDING FIRE PROTECTION SYSTEM WAS DISABLED TO PERMIT WORK IN THIS AREA, THE CONTINUOUS FIRE WATCH WAS NOT MAINTAINED IN THAT THE AREA WAS VACATED FOR AN INTERVAL OF TEN MINUTES AND THE SYSTEM WAS LEFT IN A DISARMED CONDITION. ADDITIONALLY, THESE REQUIREMENTS AS STATED IN THE LICENSEE'S "PROCEDURE FOR CONTROLLING FIREWATCH" WERE NOT ADHERED TO.
(7920 INFRACTION)

TECHNICAL SPECIFICATION 6.8.1 REQUIRES IN PART: "WRITTEN PROCEDURES AND ADMINISTRATIVE POLICIES SHALL BE ESTABLISHED, IMPLEMENTED, AND MAINTAINED...." HEALTH PHYSICS PROCEDURE HPO.CO-9C, REVISION 1, DATED DECEMBER 27, 1978, SECTION 9.0 "INVENTORY CONTROL AND PROGRAM EVALUATION" STATES IN PART, "THE LAUNDRY REPORT IS COMPLETED BY LAUNDRY PERSONNEL AND DELIVERED TO THE RESPIRATORY PROTECTION ADMINISTRATOR DAILY. THIS REPORT PROVIDES THE RESPIRATORY PROTECTION ADMINISTRATOR WITH AN INDICATION OF RESPIRATORS USED, CLEANED, REPAIRED, AND REJECTED AS WELL AS PARTS REPLACEMENT DATA TO BE USED IN PROGRAM EVALUATION." CONTRARY TO THE ABOVE, AS OF AUGUST 6, 1979, SAID REPORTS WERE BEING DELIVERED TO THE SUPERVISOR-RADIATION SUPPORT SERVICES INSTEAD OF TO THE RESPIRATORY PROTECTION ADMINISTRATOR AND NO DOCUMENTATION WAS AVAILABLE TO DEMONSTRATE PROGRAM EVALUATION.
(7920 DEFICIENCY)

10 CFR 2.790 - PHYSICAL BARRIERS - ACCESS CONTROLS.
(7920 INFRACTION)

10 CFR 50 APPENDIX B, CRITERION XVI, STATES IN PART: "MEASURES SHALL BE ESTABLISHED TO ENSURE THAT CONDITIONS ADVERSE TO QUALITY, SUCH AS FAILURES,...ARE PROMPTLY IDENTIFIED AND CORRECTED...." CONTRARY TO THE ABOVE, A CABLE TRAY PENETRATION (NOMENCLATURE Z02MV180), LOCATED IN THE UNIT 2 REACTOR BUILDING 135 FOOT ELEVATION WAS FOUND ON NOVEMBER 8,1979 TO HAVE A BROKEN SEAL WHICH COMPROMISED THE SEAL AND FIRE PROTECTION CAPABILITY. ADDITIONALLY, THE SEAL ENCLOSURE CONTAINED TRASH AND DEBRIS, CONSTITUTING A POTENTIAL FIRE HAZARD. THIS SEAL HAD LAST BEEN INSPECTED ON APRIL 23, 1979 AND NO MEASURES OR LICENSEE INSPECTION HAD IDENTIFIED AND CORRECTED THIS PENETRATION INADEQUACY.
(7929 INFRACTION)

TECHNICAL SPECIFICATION 6.1.1, "RADIATION PROTECTION PROGRAM," REQUIRES THAT PROCEDURES FOR PERSONNEL RADIATION PROTECTION SHALL BE PREPARED CONSISTENT WITH REQUIREMENTS OF 10 CFR PART 20 AND SHALL BE APPROVED, MAINTAINED, AND ADHERED TO FOR ALL OPERATIONS INVOLVING PERSONNEL EXPOSURE. HEALTH PHYSICS PROCEDURE HPO/CO-4, "RADIATION WORK PERMITS," REVISION 13, DATED MARCH 20, 1979, REQUIRES IN PART IN SECTION VI THAT ALL PERSONNEL WHO DESIRE TO ENTER THE AREA COVERED BY THE RWP SHALL READ AND BE FAMILIAR WITH THE DATA AND THE INSTRUCTIONS ON THE RWP, SIGN THEIR OWN NAME, AND PROVIDE NAME, DATE,...TIME IN, DOSIMETER READING,.... CONTRARY TO THE ABOVE REQUIREMENTS, ON NOVEMBER 8, 1979, ONE INDIVIDUAL FROM THE OPERATIONS STAFF ENTERED THE UNIT 3 165 FOOT ELEVATION TURBINE AND TURBINE HALL AREAS FOR AN INSPECTION WITHOUT SIGNING IN OR PROVIDING APPROPRIATE DATA REQUIRED BY RWP 3-I-5 GOVERNING SUCH INSPECTION ENTRIES. (RECURRENT ITEM)
(7929 INFRACTION)

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
80-002/ 01T	01/10/80	01/24/80	24 Hours	ADS nitrogen system check valves leaking and seismic supports inadequate
80-003/ 01T	01/19/80	02/01/80	24 Hours	Unit 2 and 3 offgas rad monitors out of service
80-004/ 03L	01/23/80	02/22/80	30 Day LER	Channel B flow biased rod withdrawal setpoints about 11% outside specification

INSPECTION STATUS

OTHER ITEMS OF INTEREST

FACILITY ITEMS (PLANS AND PROCEDURES):

- Refueling outage of 100 days will commence March 21, 1980.

MANAGERIAL ITEMS:

- The facility was toured by the President of INPO, E. P. Wilkinson, on February 8, 1980.

PLANT STATUS:

- The reactor operated at a nominal 100 percent during the month. Representative offgas and combined Unit 2 and Unit 3 stack gas release rates were 7690 microcuries per second and 692 microcuries per second, respectively.

- LAST IE SITE INSPECTION DATE: 2/29/80

- INSPECTION REPORT NO: 50-277/80-03

SUMMARY

INSPECTION SUMMARY

- 50-277/79-29 - Nov 1-30: Routine, onsite and corporate office regular and backshift inspections by the resident inspector (45 hours). Areas inspected included accessible portions of the facility, radiation protection, maintenance activities, physical security, plant operations, facility tours, control room inspections, housekeeping, LER review onsite and in-office, followup on previously identified items, review of periodic and special reports, and IE Bulletin followup. Noncompliances: two (failure to adhere to Radiation Work Permit (RWP) requirements; and, failure to maintain a fire and penetration seal barrier).

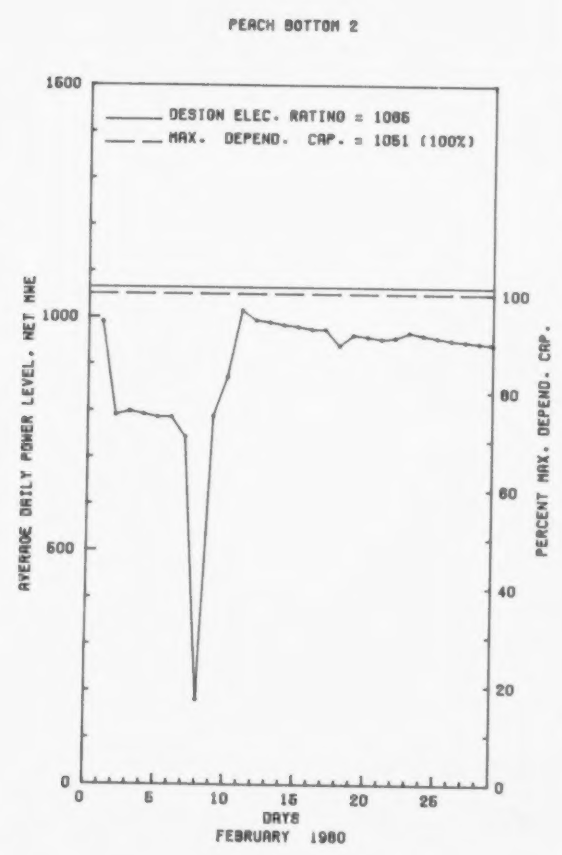
- 50-277/80-02 - Jan 8-11, 16: Routine, unannounced inspection of the Fire Protection/Prevention Program including: assignment of functional responsibilities; implementation of administrative controls; fire brigade training; observation of ignition source and combustible materials control; observation of fire protection systems and components; observation of plant modifications; review of surveillance test records associated with the fire protection program; and observation of critical fire areas. (32 hours) Noncompliances: One (failure to follow procedure for installation of fire stops).

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 4-10-80

REVIEWED BY NRR DOR: Dick Clark **DATE:** 04-11-80

PEACH BOTTOM 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-277
2. Utility Contact: W. M. Alden (215) 841-5022
3. Licensed Thermal Power (MWt): 3293
4. Nameplate Rating (Gross MWe): 1280 x 0.9 = 1152
5. Design Electrical Rating (Net MWe): 1065
6. Maximum Dependable Capacity (Gross MWe): 1098
7. Maximum Dependable Capacity (Net MWe): 1051
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	696.0	1,440.0	49,554.0
12. Hours Reactor Was Critical	696.0	1,043.0	38,798.0
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	690.2	1,004.2	37,957.0
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,951,289	2,782,416	109,548,060
17. Gross Electrical Energy (MWH)	642,440	911,120	35,962,350
18. Net Electrical Energy (MWH)	621,293	875,740	34,491,525
19. Unit Service Factor	99.2	69.7	76.6
20. Unit Availability Factor	99.2	69.7	76.6
21. Unit Capacity Factor (MDC Net)	84.9	57.9	66.2
22. Unit Capacity Factor (DER Net)	83.8	57.1	65.3
23. Unit Forced Outage Rate	.0	1.2	6.2

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUELING, 03/21/80 THIRTEEN WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
5	02/01/80	S	.0	A	5		HA	VALVEX	#1 TURBINE STOP VALVE TEST. IT FAILED TO REOPEN. POWER REDUCED TO 73%.
6	02/07/80	S	5.8	B	1		HA	VALVEX	#1 TURBINE STOP VALVE REPAIR.
7	02/09/80	S	.0	H	5		RC	ZZZZZZ	LOAD DROP FOR CONTROL ROD ADJUSTMENT.

SUMMARY

OPERATION WAS ROUTINE DURING THE REPORT PERIOD IN AN END-OF-CYCLE COASTDOWN. FROM 02/01/80 THROUGH 02/07/80, LOAD WAS REDUCED DUE TO THE TURBINE STOP VALVE NOT OPENING FOLLOWING ROUTINE TESTING. ON 02/08/80, THE TURBINE GENERATOR WAS REMOVED FROM SERVICE FOR REPAIRS. LOAD WAS ALSO REDUCED ON 02/18/80 DUE TO RIVER TEMPERATURE DIFFERENTIAL.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

PEACH BOTTOM 3

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Pennsylvania/York 19 miles S of Lancaster
2.	Electric Reliability Council	MAAC
3.	Type of Reactor	BWR
4.	Power Level (MWT)	3293
5.	Design Electrical Rating (MWE Net)	1065
6.	Date of Initial Criticality	August 7, 1974
7.	Date of Commercial Operation	December 23, 1974
8.	Condenser Cooling Method	Once Through with Cooling Towers
9.	Condenser Cooling Water Source	Conowingo Pond/Susquehanna River

Utility & Contractor Information

10.	Licensee	Philadelphia Electric Company
11.	Corporate Address	2301 Market Street Philadelphia, Pennsylvania 19101
12.	Corporate Contact	S. L. Daltroff, Vice President Electric Production
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	GE
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	IE Senior Inspector	E. McCabe
19.	IE Resident Inspector	E. Greenman (717-456-7614)
20.	Licensing Project Manager	Dick Clark
21.	Docket Number	50-278
22.	License Number/Date of Issue	DPR-56, 7/2/74
23.	Public Document Room	Government Publications Section State Library of Pennsylvania Education Building Commonwealth and Walnut Street Harrisburg, Pennsylvania 17126

INSPECTION STATUS

ENFORCEMENT STATUS

TECHNICAL SPECIFICATION 3.14.B.3, "CO2 FIRE PROTECTION SYSTEMS" STATES, "THE DIESEL GENERATOR CO2 FIRE PROTECTION SHALL BE OPERABLE WHEN THE DIESEL GENERATORS ARE REQUIRED TO BE OPERABLE WITH THE SYSTEM COMPRISED OF: ...B. AN OPERABLE FLOW PATH TO EACH ROOM." TECHNICAL SPECIFICATION 3.14.13.4 STATES IN PART, "IF THE REQUIREMENTS OF 3.14.B.1, 2, OR 3 CANNOT BE MET, (A.) ESTABLISH A CONTINUOUS FIRE WATCH...FOR THE UNPROTECTED AREA (HPCI, CABLE SPREADING, COMPUTER, DIESEL GENERATOR) WITHIN ONE HOUR." CONTRARY TO THE ABOVE, ON AUGUST 21, 1979, WITH UNITS 2 AND 3 OPERATING IN THE RUN MODE AND DIESEL GENERATOR OPERABILITY REQUIRED PURSUANT TO TECHNICAL SPECIFICATIONS 3.9.A AND 3.9.B, AFTER PROPERLY ESTABLISHING A REQUIRED FIRE WATCH WHILE A PORTION OF THE DIESEL GENERATOR BUILDING FIRE PROTECTION SYSTEM WAS DISABLED TO PERMIT WORK IN THIS AREA, THE CONTINUOUS FIRE WATCH WAS NOT MAINTAINED IN THAT THE AREA WAS VACATED FOR AN INTERVAL OF TEN MINUTES AND THE SYSTEM WAS LEFT IN A DISARMED CONDITION. ADDITIONALLY, THESE REQUIREMENTS AS STATED IN THE LICENSEE'S "PROCEDURE FOR CONTROLLING FIREWATCH" WERE NOT ADHERED TO.
(7922 INFRACTION)

TECHNICAL SPECIFICATION 6.8.1 STATES IN PART, "WRITTEN PROCEDURES AND ADMINISTRATIVE POLICIES SHALL BE ESTABLISHED, IMPLEMENTED, AND MAINTAINED...." PROCEDURE NO. S.3.6.E, "STANDBY LIQUID CONTROL SYSTEM OPERABILITY TEST," REVISION 6, DATED JULY 12, 1978, STEP 32 STATES, "COMPLETE A NEW SLC SYSTEM C.O.L. S.3.6.A AND RETURN TO SHIFT SUPERVISION FOR REVIEW AND FILING." CHECK-OFF LIST (C.O.L.) S.3.6.A, "STANDBY LIQUID CONTROL SYSTEM M-358," REVISION 9, DATED AUGUST 16, 1979, STEP 15 STATES, "TEST TANK DRAIN VALVE (28) - CLOSED." STEP 17 STATES, "TEST TANK DEMINERALIZER WATER HOSE CONNECTION (29) - CLOSED." CONTRARY TO THE ABOVE, PROCEDURES WERE NOT IMPLEMENTED IN THAT AT APPROXIMATELY 2:00 P.M. ON AUGUST 22, 1979, VALVE NUMBERS 28 AND 29 OF THE STANDBY LIQUID CONTROL SYSTEM WERE FOUND OPEN WHILE THE MOST RECENT C.O.L. S.3.6.A, COMPLETED ON AUGUST 21, 1979, INDICATED THAT THE VALVES WERE CLOSED. (UNIT 3)
(7922 INFRACTION)

TECHNICAL SPECIFICATION 6.8.1 REQUIRES IN PART: "WRITTEN PROCEDURES AND ADMINISTRATIVE POLICIES SHALL BE ESTABLISHED, IMPLEMENTED, AND MAINTAINED...." HEALTH PHYSICS PROCEDURE HPO/CO-9C, REVISION 1, DATED DECEMBER 27, 1978, SECTION 9.0 "INVENTORY CONTROL AND PROGRAM EVALUATION" STATES IN PART, "THE LAUNDRY REPORT IS COMPLETED BY LAUNDRY PERSONNEL AND DELIVERED TO THE RESPIRATORY PROTECTION ADMINISTRATOR DAILY. THIS REPORT PROVIDES THE RESPIRATORY PROTECTION ADMINISTRATOR WITH AN INDICATION OF RESPIRATORS USED, CLEANED, REPAIRED, AND REJECTED AS WELL AS PARTS REPLACEMENT DATA TO BE USED IN PROGRAM EVALUATION." CONTRARY TO THE ABOVE, AS OF AUGUST 6, 1979, SAID REPORTS WERE BEING DELIVERED TO THE SUPERVISOR-RADIATION SUPPORT SERVICES INSTEAD OF TO THE RESPIRATORY PROTECTION ADMINISTRATOR AND NO DOCUMENTATION WAS AVAILABLE TO DEMONSTRATE PROGRAM EVALUATION.
(7922 DEFICIENCY)

TECHNICAL SPECIFICATION 6.1.1, "RADIATION PROTECTION PROGRAM," REQUIRES THAT PROCEDURES FOR PERSONNEL RADIATION PROTECTION SHALL BE PREPARED CONSISTENT WITH REQUIREMENTS OF 10 CFR PART 20 AND SHALL BE APPROVED, MAINTAINED, AND ADHERED TO FOR ALL OPERATIONS INVOLVING PERSONNEL EXPOSURE. HEALTH PHYSICS PROCEDURE HPO/CO-4, "RADIATION WORK PERMITS," REVISION 13, DATED MARCH 20, 1979, REQUIRES IN PART IN SECTION VI THAT ALL PERSONNEL WHO DESIRE TO ENTER THE AREA COVERED BY THE RWP SHALL READ AND BE FAMILIAR WITH THE DATA AND THE INSTRUCTIONS ON THE RWP, SIGN THEIR OWN NAME, AND PROVIDE NAME, DATE....TIME IN, DOSIMETER READING..... CONTRARY TO THE ABOVE REQUIREMENT, ON NOVEMBER 8, 1979, ONE INDIVIDUAL FROM THE OPERATIONS STAFF ENTERED THE UNIT 3 165 FOOT ELEVATION TURBINE AND TURBINE HALL AREAS FOR AN INSPECTION WITHOUT SIGNING IN OR PROVIDING APPROPRIATE DATA REQUIRED BY RWP 3-I-5 GOVERNING SUCH INSPECTION ENTRIES. (RECURRENT ITEM).
(7932 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

- A recombiner hydrogen analyzer leaked while being placed in service on 2/15, resulting in an unplanned release of 1.3 curies via the reactor building ventilation system. The maximum release rate was one-third of the allowable Tech Specs limit. The analyzer was isolated, repaired and returned to service the same day.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
80-001/ 03L	01/06/80	02/05/80	30 Day LER	Main steam line radiation monitor "D" loss of trip function discovered during routine surveillance
80-002/ 03L	01/18/80	02/15/80	30 Day LER	One traversing incore probe (TIP) "in shield" limit switch was out of calibration such that in the event of a PCIS signal the ball valve would not isolate the Tip tube
80-003/ 01T	01/19/80	02/01/80	24 Hour	Unit 2 and 3 offgas rad monitors out of service

MANAGERIAL ITEMS:

- The facility was toured by the President of INPO, E. P. Wilkinson, on February 8, 1980.

PLANT STATUS:

- Unit was online 2/6 following an eight day outage. The reactor was operated at a nominal 100 percent power. Representative offgas and combined Unit 2 and Unit 3 stack gas release rates were 3710 microcuries per second and 692 microcuries per second, respectively.

- LAST IE SITE INSPECTION DATE: 2/29/80

- INSPECTION REPORT NO: 50-278/80-03

SUMMARY

INSPECTION SUMMARY

- 50-278/79-32 - Nov 1-30: Routine, onsite and corporate office regular and backshift inspections by the resident inspector (46 hours). Areas inspected included system restoration from a refueling outage, startup, shutdown margin testing, accessible portions of the Unit 2 and Unit 3 facilities, radiation protection, maintenance activities, physical security, plant operations, facility tours, control room inspections, housekeeping, LER review onsite and in-office, followup on previously identified items, review of periodic and special reports, and IE Bulletin followup. Noncompliances: Two (failure to adhere to Radiation Work Permit (RWP) requirements; and, failure to maintain a fire and penetration seal barrier).

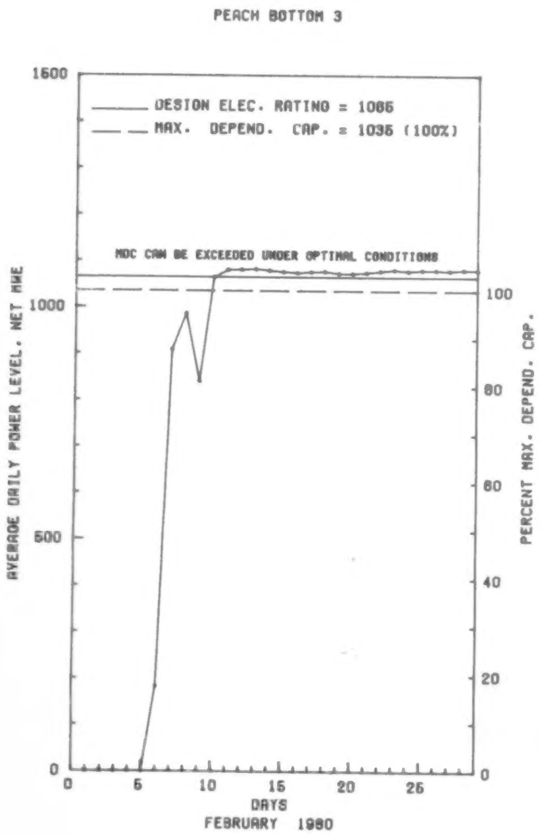
- 50-278/80-02 - Jan 8-11, 16: Routine, unannounced inspection of the Fire Protection/Prevention Program including: assignment of functional responsibilities; implementation of administrative controls; fire brigade training; observation of ignition source and combustible materials control; observation of fire protection systems and components; observation of plant modifications; review of surveillance test records associated with the fire protection program; and observation of critical fire areas. (32 hours) Noncompliances: One (failure to follow procedure for installation of fire-stops).

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 4-10-80

REVIEWED BY NRR DOR: Dick Clark **DATE:** 4-11-80

PEACH BOTTOM 3

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-278
2. Utility Contact: W. M. Alden (215) 841-5022
3. Licensed Thermal Power (MWt): 3293
4. Nameplate Rating (Gross MWe): 1280 x 0.9 = 1152
5. Design Electrical Rating (Net MWe): 1065
6. Maximum Dependable Capacity (Gross MWe): 1098
7. Maximum Dependable Capacity (Net MWe): 1035
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours In Reporting Period	696.0	1,440.0	45,408.0
12. Hours Reactor Was Critical	588.0	1,153.0	35,999.0
13. Reactor Reserve Shutdown Hours	.0	.0	0
14. Hours Generator On-Line	561.1	1,101.8	35,033.6
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,795,642	3,502,918	97,722,072
17. Gross Electrical Energy (MWH)	604,670	1,178,730	31,764,590
18. Net Electrical Energy (MWH)	586,452	1,139,911	30,484,711
19. Unit Service Factor	80.6	76.5	77.0
20. Unit Availability Factor	80.6	76.5	77.0
21. Unit Capacity Factor (MDC Net)	81.4	76.5	64.8
22. Unit Capacity Factor (DER Net)	79.1	74.3	62.9
23. Unit Forced Outage Rate	.0	0	6.7

24. Shutdowns Scheduled Over Next 6 Months (Type,Date,and Duration of Each):
REFUELING, 03/07/81, ELEVEN WEEKS
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
3	01/29/80	S	134.9	B	4		CB	PUMPXX	3B REACTOR RECIRC. PUMP SEAL REPLACEMENT.
4	02/09/80	S	.0	H	5		RC	ZZZZZZ	LOAD DROP FOR CONTROL ROD ADJUSTMENT.

SUMMARY

OPERATION WAS AT NEAR FULL POWER FOLLOWING RETURN TO SERVICE ON 02/06/80 FROM AN OUTAGE TO REPLACE THE 3B RECIRCULATION PUMP SEAL AND FOR SEISMIC INSPECTION OF SOME INSTRUMENT LINES. BRIEF LOAD REDUCTIONS HAVE ACCURRED ON 02/07/80 WHEN A RUNBACK OF THE TURBINE-GENERATOR AND RECIRCULATION PUMPS RESULTED FROM A REACTOR LEVEL TRANSIENT ASSOCIATED WITH TESTING THE "B" REACTOR PUMP, AND ON 02/10/80 FOR CONTROL ROD ADJUSTMENT.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

PILGRIM

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Massachusetts/Plymouth 4 miles SE of Plymouth
2.	Electric Reliability Council	NPCC
3.	Type of Reactor	BWR
4.	Licensed Power Level (MWT)	1998
5.	Design Electrical Rating (MWE Net)	655
6.	Date of Initial Criticality	June 16, 1972
7.	Date of Commercial Operation	December 1972
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Cape Cod Bay

Utility & Contractor Information

10.	Licensee	Boston Edison Company
11.	Corporate Address	800 Boylston Street Boston, Massachusetts 02199
12.	Corporate Contact	A. Corey, Vice President
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	GE
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	IE Senior Inspector	E. McCabe
19.	IE Principal Inspector	K. Roberts
20.	Licensing Project Manager	J. Hannon
21.	Docket Number	50-293
22.	License Number/Date of Issue	DPR-35, 6/8/72
23.	Public Document Room	Plymouth Public Library North Street Plymouth, Massachusetts 02360

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS:

♦ Shutdown for refueling until about mid-April.

♦ LAST IE SITE INSPECTION DATE: 3/7/80

♦ INSPECTION REPORT NO: 50-293/80-12

SUMMARY

INSPECTION SUMMARY

♦ 50-293/79-07 - Oct 9-10: Special, unannounced inspection by two region-based inspectors (17 hr. onsite) of the licensee's radiation protection program during operations. Areas inspected included training, surveys, posting and control, and facility tours. No noncompliances were observed.

♦ 50-293/79-23 - Dec 27-28 & Jan 4: Special unannounced inspection by two region-based inspectors of the licensee's discrepant radioactive waste shipment to Barnwell, S.C. on December 14, 1979 including: records of shipments, review of procedures, chronology of events, and regulatory requirements. One item of noncompliance was identified (failure to comply with 40 CFR 183.393(j)(3)).

♦ 50-293/80-04 - Jan 28-31: Routine, unannounced inspection by one region-based and one resident inspector of ISI implementation, Snubber Program, the Containment Integrated Leak Rate Test Procedure and plant tours. The inspection involved 64 hours onsite. No items of noncompliance were identified.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-051/ 03L	12/31/79	01/24/80	30 Day LER	Temporary loss of reactor coolant leak detection air sampling system
80-001/ 01T	01/22/80	02/01/80	Special	Identification of deficient areas with respect to fire protection measures as they relate to safe shutdown capability
80-002/ 01T	01/24/80	02/05/80	Special	HPCI and ADS cannot meet design criteria for cable separation
80-003/ 03L	01/08/80	02/01/80	30 Day LER	Excessive seat leakage past vacuum breaker AO 5040A (T.S. 8.7.1.5 limit exceeded)
80-004/ 01T	02/01/80	02/15/80	24 Hour LER	Cracks observed on core spray spargers
80-005/ 04T	01/31/80	02/06/80	14 Day LER	Milk sample greater than 10 times lower limit of detection (ST-90)
80-006/ 03L	01/15/80	02/07/80	30 Day LER	Snubber strut bushing not welded as required
80-007/ 03L	01/20/80	02/18/80	30 Day LER	Spurious fire alarm caused by maintenance in area of detector

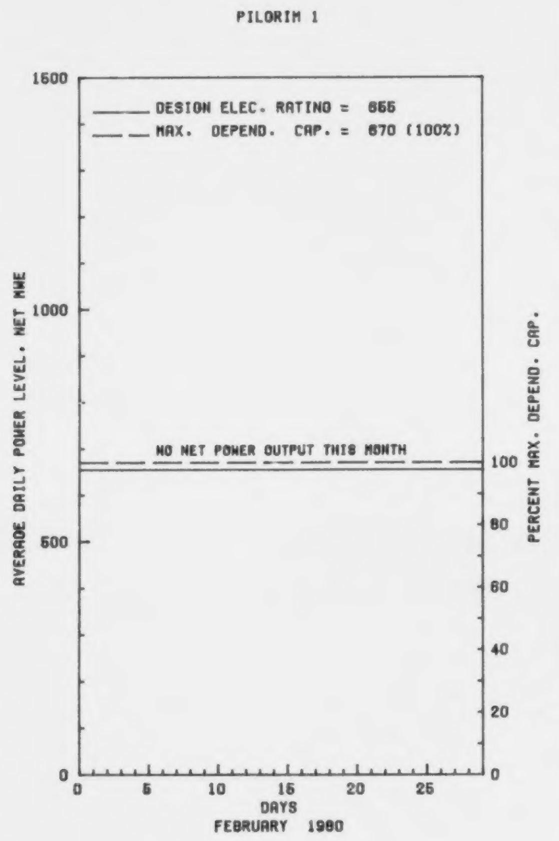
REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 4-10-80

REVIEWED BY NRR DOR: John N. Hannon **DATE:** 4-14-80

best copy available

PILGRIM

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-293
2. Utility Contact: G. G. Whitney (617) 746-7900
3. Licensed Thermal Power (MWt): 1998
4. Nameplate Rating (Gross MWe): 780 x 0.87 = 678
5. Design Electrical Rating (Net MWe): 655
6. Maximum Dependable Capacity (Gross MWe): 690
7. Maximum Dependable Capacity (Net MWe): 670
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	696.0	1,440.0	63,336.0
12. Hours Reactor Was Critical	.0	96.1	45,077.0
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	.0	96.1	43,648.6
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	181,152	73,272,264
17. Gross Electrical Energy (MWH)	0	61,340	24,226,174
18. Net Electrical Energy (MWH)	0	58,860	23,264,983
19. Unit Service Factor	.0	6.7	68.9
20. Unit Availability Factor	.0	6.7	68.9
21. Unit Capacity Factor (MDC Net)	.0	6.1	54.8
22. Unit Capacity Factor (DER Net)	.0	6.2	56.1
23. Unit Forced Outage Rate	.0	.0	10.6

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 05/15/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
1	01/05/80	S	696.0	C	4		RC	FUELXX	REFUELING CONTINUES.

SUMMARY

THE UNIT REMAINED SHUTDOWN THROUGHOUT THE REPORT PERIOD FOR REFUELING.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

POINT BEACH 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Wisconsin/Manitowoc 15 miles N of Manitowoc
2.	Electric Reliability Council	
3.	Type of Reactor	PWR
4.	Power Level (MWT)	1518
5.	Design Electrical Rating (MWE Net)	497
6.	Date of Initial Criticality	November 2, 1970
7.	Date of Commercial Operation	December 21, 1970
8.	Condenser Cooling Method	Straight Through
9.	Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10.	Licensee	Wisconsin Electric Power Co.
11.	Corporate Address	231 West Michigan Milwaukee, Wisconsin
12.	Corporate Contact	S. Burstein
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Senior Inspector	R. F. Warnick
19.	IE Resident Inspector	B.L. Jorgnesen
20.	Licensing Project Manager	C. M. Trammell
21.	Docket Number	50-266
22.	License Number/Date of Issue	DPR-24 issued 10/05/70
23.	Public Document Room	Document Department, Library University of Wisconsin-Stevens Point Stevens Point, Wisconsin 54481 Manitowoc Public Library Manitowoc, Wisconsin (Temporary LPDR for spent fuel pool capacity increase amendment only

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

+ The Unit operated at approximately 77% power this month. This power level is about the maximum obtainable due to reduced primary temperature and pressure limits. Per the conditions of the order modifying the conditions of the license, Unit 1 was taken off line for inservice inspection of the steam generator tubes on February 28, 1980.

\+LAST IE SITE INSPECTION DATE: December 11-12, 1979

+ INSPECTION REPORT NO: 79-20

SUMMARY

INSPECTION SUMMARY

+ Inspection on October 11; November 14-15; December 17-19, 1979 and January 7-8, 1980, (79-14): QA/QC program, implementing procedures, observation of work activities, and review and evaluation of nondestructive examination (NDE) data and reports of inservice inspection (ISI); IE Bulletin 79-13 and IE Bulletin 79-17 activities; and steam generator repair. The inspection involved 25 inspector hours onsite and five hours at southwest Research Institute by one NRC inspector. No items of noncompliance or deviations were identified.

+ Inspection on December 4-7, 14-15 and 18, (79-19): Routine, unannounced inspection on December 4-7 to review plant operations, procedures, bulletin follow-up, non-routine event follow-up and independent inspection. Announced inspection on December 14-15 and 18 for non-routine event follow-up. The inspection involved 41 inspector hours onsite by one NRC inspector. Of the four areas inspected, no items of noncompliance were identified.

+ Inspection on December 11-12, (79-20): Nonroutine, announced inspection of licensee actions taken in response to IE Bulletin 79-19, "Packaging of Low-Level Radioactive Waste for Transport and Burial." The inspection involved 13 inspector hours onsite by one NRC inspector. No items of noncompliance or deviations were identified.

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
80-01/ 01T-0	02/02/80	02/15/80	14 Day LER	Pressure transmitters isolation block closed.

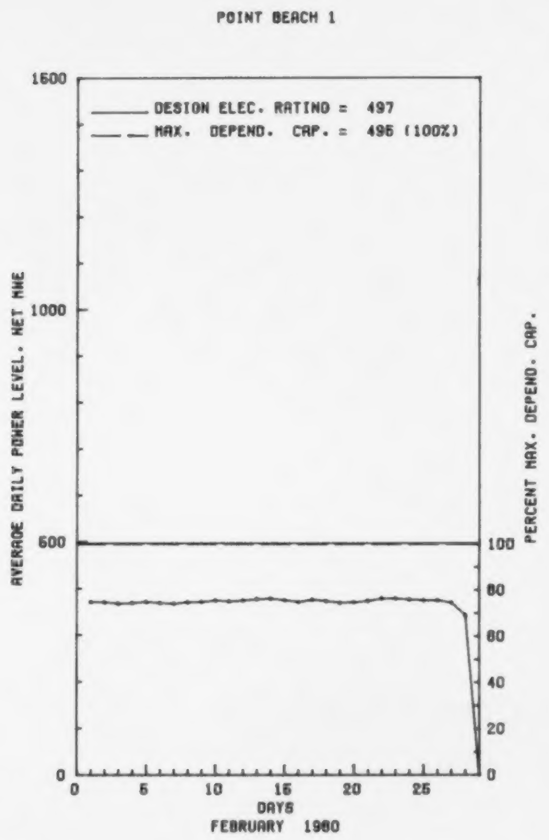
===

REVIEWED BY OMPA LOEB: T.C. Cintula DATE: 4·10·80

REVIEWED BY NRR DOR: C M Trammell DATE: 4-11-80

POINT BEACH 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

ITEM 21 CALCULATED BY USING A WEIGHTED AVERAGE

1. Docket Number: 50-266
2. Utility Contact: C.W. FAY (414) 277-2811
3. Licensed Thermal Power (MWt): 1518
4. Nameplate Rating (Gross MWe): 582 x 0.9 = 524
5. Design Electrical Rating (Net MWe): 497
6. Maximum Dependable Capacity (Gross MWe): 519
7. Maximum Dependable Capacity (Net MWe): 495
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): 390
10. Reasons for Restrictions, If Any:
ATTEMPT TO INHIBIT CORROSION OF STEAM GENERATOR TUBES.

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	696.0	1,440.0	81,672.0
12. Hours Reactor Was Critical	685.7	1,429.7	67,946.2
13. Reactor Reserve Shutdown Hours	10.3	10.3	592.5
14. Hours Generator On-Line	671.7	1,415.7	65,809.7
15. Unit Reserve Shutdown Hours	24.3	24.3	626.7
16. Gross Thermal Energy (MWH)	811,187	1,696,614	92,449,409
17. Gross Electrical Energy (MWH)	263,090	554,070	31,187,820
18. Net Electrical Energy (MWH)	249,152	525,243	29,715,421
19. Unit Service Factor	96.5	98.3	80.5
20. Unit Availability Factor	100.0	100.0	81.3
*21. Unit Capacity Factor (MDC Net)	72.3	73.7	74.8
22. Unit Capacity Factor (DER Net)	72.0	73.4	73.2
23. Unit Forced Outage Rate	.0	.0	3.6

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 03/21/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
1	02/28/80	S	24.3	D	1		CB	HTEXCH	THE UNIT WAS TAKEN OFF LINE IN ACCORDANCE WITH AN NRC CONFIRMATORY ORDER OF NOVEMBER 30, 1979 WHICH REQUIRED PERFORMANCE OF STEAM GENERATOR LEAK TESTING AND EDDY CURRENT INSPECTION. UNIT 1 INSPECTIONS WILL BEGIN UPON COMPLETION OF SIMILAR WORK CURRENTLY BEING PERFORMED ON UNIT 2.

SUMMARY

OPERATION WAS ROUTINE DURING THE REPORT PERIOD AT A SELF-IMPOSED RESTRICTION TO 390 MWE (NET) IN AN ATTEMPT TO INHIBIT CORROSION OF STEAM GENERATOR TUBES WITHIN THE TUBESHEET CREVICE. THE UNIT WAS TAKEN OFF LINE ON 02/28/80 FOR STEAM GENERATOR LEAK CHECKS AND EDDY CURRENT INSPECTION.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

POINT BEACH 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Wisconsin/Manitowoc 15 miles N of Manitowoc
2.	Electric Reliability Council	
3.	Type of Reactor	PWR
4.	Power Level (MWT)	1518
5.	Design Electrical Rating (MWE Net)	497
6.	Date of Initial Criticality	May 30, 1972
7.	Date of Commercial Operation	October 1, 1972
8.	Condenser Cooling Method	Straight Through
9.	Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10.	Licensee	Wisconsin Electric Power Co.
11.	Corporate Address	231 West Michigan Milwaukee, Wisconsin
12.	Corporate Contact	S. Burstein
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Senior Inspector	R. F. Warnick
19.	IE Resident Inspector	B.L. Jorgnesen
20.	Licensing Project Manager	C. M. Trammell
21.	Docket Number	50-301
22.	License Number/Date of Issue	DPR-27 issued 11/16/71
23.	Public Document Room	Document Department, Library University of Wisconsin-Stevens Point Stevens Point, Wisconsin 54481 Manitowoc Public Library Manitowoc, Wisconsin (Temporary LPDR for spent fuel pool capacity increase amendment only

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

+ The plant had an unplanned shutdown this month due to steam generator tube leak on February 28, 1980. Repairs should be completed approximately March 11, 1980.

+LAST IE SITE INSPECTION DATE: December 11-12, 1979

+ INSPECTION REPORT NO: 79-22

SUMMARY

INSPECTION SUMMARY

+ Inspection on October 11; November 14-15; December 17-19, 1979 and January 7-8, 1980, (79-16): QA/QC program, implementing procedures, observation of work activities, and review and evaluation of nondestructive examination (NDE) data and reports of inservice inspection (ISI); IE Bulletin 79-13 and IE Bulletin 79-17 activities; and steam generator repair. The inspection involved 25 inspector hours onsite and five hours at southwest Research Institute by one NRC inspector. No items of noncompliance or deviations were identified.

+ Inspection on December 4-7, 14-15 and 18, (79-21): Routine, unannounced inspection on December 4-7 to review plant operations, procedures, bulletin follow-up, non-routine event follow-up and independent inspection. Announced inspection on December 14-15 and 18 for non-routine event follow-up. The inspection involved 41 inspector hours onsite by one NRC inspector. Of the four areas inspected, no items of noncompliance were identified.

+ Inspection on December 11-12, (79-22): Nonroutine, announced inspection of licensee actions taken in response to IE Bulletin 79-19, "Packaging of Low-Level Radioactive Waste for Transport and Burial." The inspection involved 13 inspector hours onsite by one NRC inspector. No items of noncompliance or deviations were identified.

REPORTS RECEIVED FROM LICENSEE

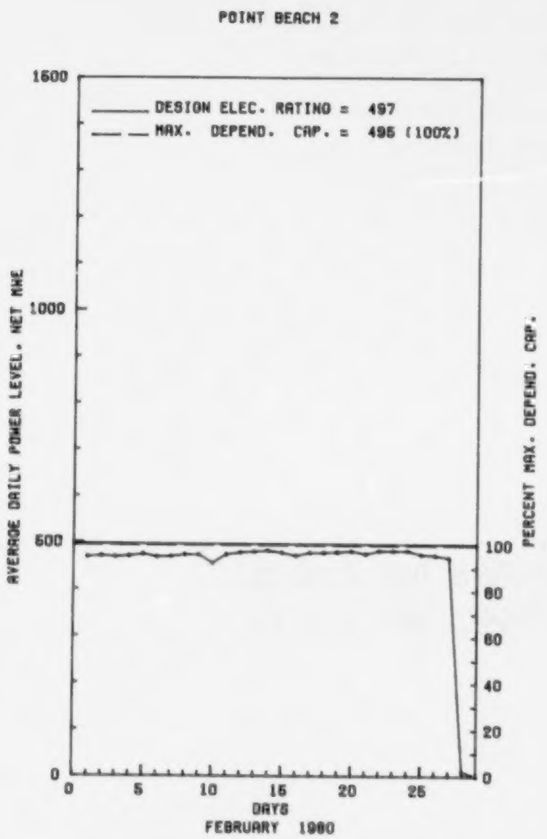
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
80-02/ 01T-0	02/27/80	02/28/80	14 Day LER	"A" SG primary to secondary leakage.

REVIEWED BY OMPA LOEB: T.C. Cintula **DATE:** 4·10·80

REVIEWED BY NRR DOR: CM Trammell **DATE:** 4-11-80

POINT BEACH 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-301
2. Utility Contact: C.W. FAY (414) 277-2811
3. Licensed Thermal Power (MWt): 1518
4. Nameplate Rating (Gross MWe): 582 x 0.9 = 524
5. Design Electrical Rating (Net MWe): 497
6. Maximum Dependable Capacity (Gross MWe): 519
7. Maximum Dependable Capacity (Net MWe): 495
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

ITEM 21 CALCULATED BY USING A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	696.0	1,440.0	66,457.0
12. Hours Reactor Was Critical	650.4	1,394.4	60,291.3
13. Reactor Reserve Shutdown Hours	.0	.0	166.1
14. Hours Generator On-Line	650.4	1,394.4	59,121.4
15. Unit Reserve Shutdown Hours	.0	.0	106.5
16. Gross Thermal Energy (MWH)	974,189	2,085,691	79,738,227
17. Gross Electrical Energy (MWH)	322,807	692,500	27,353,840
18. Net Electrical Energy (MWH)	307,968	661,372	25,736,499
19. Unit Service Factor	93.4	96.8	89.0
20. Unit Availability Factor	93.4	96.8	89.1
*21. Unit Capacity Factor (MDC Net)	89.4	92.8	78.0
22. Unit Capacity Factor (DER Net)	89.0	92.4	77.9
23. Unit Forced Outage Rate	6.6	3.2	1.5

24. Shutdowns Scheduled Over Next 6 Months (Type,Date,and Duration of Each): REFUELING - 04/11/80 - 5 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: 03/13/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
1	02/28/80	F	45.6	A	1	80-002	CB	HTEXCH	SHUTDOWN OF THE UNIT ON FEBRUARY 28, 1980, FOLLOWING CONFIRMATION OF PRIMARY-TO-SECONDARY LEAKAGE OF 1,420 GALLONS PER DAY IN THE "A" STEAM GENERATOR. A STATIC HEAD LEAK CHECK IDENTIFIED THE DEFECTIVE TUBE AND SUBSEQUENT EDDY CURRENT INSPECTION PLACED THE DEFECT AT 10" ABOVE THE TUBE END ON THE HOT LEG SIDE.

SUMMARY

OPERATION WAS ROUTINE DURING THE REPORT PERIOD, WITH AN END-OF-CYCLE COASTDOWN BEGINNING 02/20/80. ON 02/28/80, THE UNIT WAS SHUTDOWN DUE TO PRIMARY-TO-SECONDARY **LEAKAGE OF 1,420 GALLONS/DAY IN THE "A" STEAM** GENERATOR. PREVIOUSLY SCHEDULED REFUELING AND EDDY CURRENT INSPECTION HAVE BEEN INCLUDED IN THIS OUTAGE. BRIEF LOAD REDUCTIONS OCCURRED ON 02/10/80 FOR TURBINE STOP VALVE TESTING, AND ON 02/21/80 BECAUSE OF TURBINE RUNBACK CAUSED BY A MOMENTARY LOSS OF POWER ON THE REAL INSTRUMENT BUS.

LEGEND

[1] Type:
F-Forced
S-Scheduled

[2] Reason:
A-Equipment Failure
B-Maintenance or Test
C-Refueling
D-Regulatory Restrictio
E-Operator Training & License Examination
F-Administrative
G-Operational Error
H-Other

[3] Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Continuations
5-Load Reduction
9-Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

PRAIRIE ISLAND 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Minnesota/Goodhue 28 miles SE of Minneapolis
2.	Electric Reliability Council	MARCA
3.	Type of Reactor	PWR
4.	Power Level (MWT)	1650
5.	Design Electrical Rating (MWE Net)	530
6.	Date of Initial Criticality	December 1, 1973
7.	Date of Commercial Operation	December 16, 1973
8.	Condenser Cooling Method	Cooling Towers
9.	Condenser Cooling Water Source	Mississippi River

Utility & Contractor Information

10.	Licensee	Northern States Power Company
11.	Corporate Address	414 Nicollet Mall - 8th Floor Minneapolis, Minnesota 55401
12.	Corporate Contact	G. H. Neils (Oper.) L. O. Mayer (Lic.)
13.	Architect/Engineer	Pioneer Services & Eng. Co.
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Northern States Power Co.

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Senior Inspector	R. F. Warnick
19.	IE Resident Inspector	C. D. Feierabend 612-388-8209
20.	Licensing Project Manager	M. Grotenhuis
21.	Docket Number	50-282
22.	License Number/Date of Issue	DPR-42 issued 8/9/73
23.	Public Document Room	Environmental Conservation Library Minneapolis Public Library 300 Nicollet Mall Minneapolis, Minnesota 55401

INSPECTION STATUS

ENFORCEMENT STATUS

ENVIRONMENTAL TECHNICAL SPECIFICATION 2.4.3 REQUIRES THAT THE CONTENTS OF THE NEUTRALIZING TANK SHALL BE CONTROLLED SO THAT DISCHARGES FROM THE TANK SHALL HAVE A PH VALUE OF NOT LESS THAN 6.5 NOR GREATER THAN 8.5. CONTRARY TO THE ABOVE REQUIREMENT, ON OCTOBER 7, 1979, A NEUTRALIZING TANK BATCH WAS RELEASED WITH A PH OF 11.65.
(7932 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

Operating routinely.

+LAST IE SITE INSPECTION DATE: January 8-10, 1980

+ INSPECTION REPORT NO: 80-02

SUMMARY

INSPECTION SUMMARY

- Inspection on October 6-10 and January 3, 1980, (79-25): QA/QC programs, implementing procedures, work activities, and records relative to steam generator tube inspections. The inspection involved 46 inspector hours onsite by one NRC inspector. No items of noncompliance or deviations were identified.
- Inspection on December 1-31, (79-30): Routine resident inspection of plant operations, maintenance, security, review and audit, calibration, radiation protection, radioactive waste system, followup of IE bulletins and circulars. The inspection involved 71 inspector hours onsite by the resident inspector. No items of noncompliance or deviations were identified.
- Inspection on December 19-21, (79-32): Routine, announced inspection of confirmatory measurements for Unit 1, including the licensee's program for quality control of analytical measurements; evaluation of the licensee's analytical results of plant effluent samples obtained during a previous inspection; collection of plant effluent samples for future comparative analysis; and submission of a spiked particulate filter, charcoal adsorber and liquid sample for future comparative analyses; and environmental protection, including administrative and procedural control system; quality assurance and control in analytical measurements; implementation of the radiological environmental monitoring program (REMP); review of Environmental Event Reports (EER), and licensee actions on previously identified matters. Of the 10 areas inspected, no apparent items of noncompliance or deviations were identified in nine areas; one apparent item of noncompliance was identified in one area (neutralizing tank batch release with pH greater than 8.5).
- Inspection on January 8-10, (80-02): Routine, unannounced inspection of the radiation protection program for operating and refueling, including: qualifications; licensee audits; training; radiation protection procedures; advance planning and preparation for refueling; instruments and equipment; exposure control; posting, labeling, and control; notification and reports; IE Bulletin 79-19; and surveys. The inspection involved 25 inspector hours onsite by one NRC inspector. Of the 11 areas inspected, no items of noncompliance or deviations were identified.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

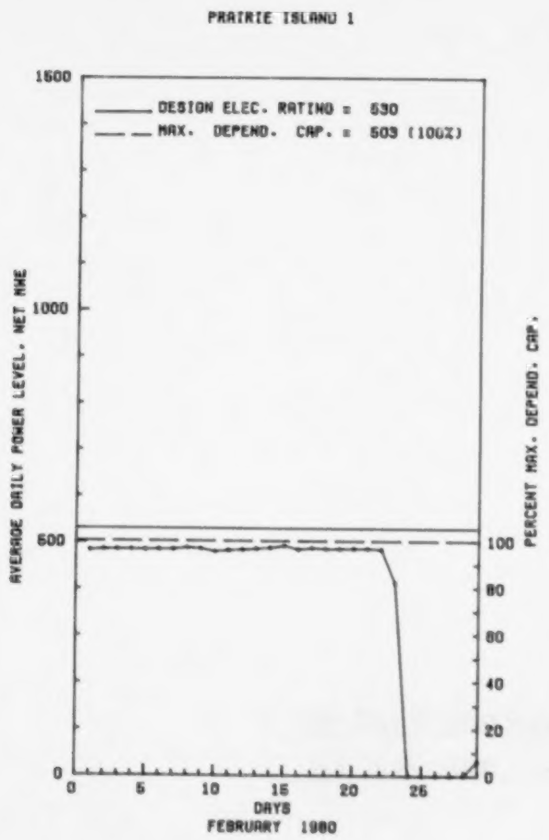
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
80-01/ 01T-0	01/06/80	01/18/80	14 Day LER	Both Unit No. 1 caustic addition valves CA-1-2 and CA-1-4, were mistakenly closed and secure cards attached.
80-02/ 03L-0	01/07/80	02/06/80	30 Day LER	During surveillance test of D1 Diesel Generator, a relay misoperation occurred which would have prevented automatic synchronization of the gener. onto Bus 15.
80-04/ 03L-0	01/18/80	02/15/80	30 Day LER	Main steam flow transmitter FT-465 was found to have drifted such that reactor trip input and steam line isolation was out of Tech. Spec. limits.
80-05/ 03L-0	01/21/80	02/20/80	30 Day LER	A lockout occurred while the engine was being prelubed.
80-06/ 01T-0	[illegible]12/80	02/06/80	14 Day LER	During operability testing of D2 Diesel Generator, the ability to reduce generator load from control room was lost.

REVIEWED BY OMPA LOEB: T.C. Cintula **DATE:** 4-10-80

REVIEWED BY NRR DOR: W. J. Ross **DATE:** 4-11-80

PRAIRIE ISLAND 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-282
2. Utility Contact: A. A. Hunstad (612) 388-4767
3. Licensed Thermal Power (MWt): 1650
4. Nameplate Rating (Gross MWe): 659 x 0.9 = 593
5. Design Electrical Rating (Net MWe): 530
6. Maximum Dependable Capacity (Gross MWe): 534
7. Maximum Dependable Capacity (Net MWe): 503
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours In Reporting Period	696.0	1,440.0	54,408.0
12. Hours Reactor Was Critical	564.9	1,308.9	42,517.3
13. Reactor Reserve Shutdown Hours	.0	.0	5,531.9
14. Hours Generator On-Line	559.8	1,303.8	41,627.4
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	905,162	2,119,654	64,198,568
17. Gross Electrical Energy (MWH)	284,920	669,840	20,823,510
18. Net Electrical Energy (MWH)	266,528	627,776	19,467,029
19. Unit Service Factor	80.4	90.5	76.1
20. Unit Availability Factor	80.4	90.5	76.1
21. Unit Capacity Factor (MDC Net)	76.1	86.7	71.1
22. Unit Capacity Factor (DER Net)	72.3	82.3	67.5
23. Unit Forced Outage Rate	.0	.0	13.2

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
REFUELING, SUMMER 1980, 5 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	02/23/80	S	136.2	B	1				SHUTDOWN TO REPAIR NO 11 REACTOR COOLANT PUMP NO. 2 SEAL.

SUMMARY

OPERATION WAS ROUTINE DURING THE REPORT PERIOD, EXCEPT FOR A SHUTDOWN FROM 02/23/80 THROUGH 02/28/80 TO REPAIR THE NO. 11 REACTOR COOLANT PUMP NO. 2 SEAL.

LEGEND

[1] Type:
F–Forced
S–Scheduled

[2] Reason:
A–Equipment Failure
B–Maintenance or Test
C–Refueling
D–Regulatory Restriction
E–Operator Training & License Examination
F–Administrative
G–Operational Error
H–Other

[3] Method:
1–Manual
2–Manual Scram
3–Automatic Scram
4–Continuations
5–Load Reduction
9–Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

PRAIRIE ISLAND 2

FACILITY DATA

Facility Description

1. Location (state/county) (nearest population center)	Minnesota/Goodhue 28 miles SE of Minneapolis
2. Electric Reliability Council	MARCA
3. Type of Reactor	PWR
4. Power Level (MWT)	1650
5. Design Electrical Rating (MWE Net)	530
6. Date of Initial Criticality	December 17, 1974
7. Date of Commercial Operation	December 21, 1974
8. Condenser Cooling Method	Cooling Towers
9. Condenser Cooling Water Source	Mississippi River

Utility & Contractor Information

10. Licensee	Northern States Power Company
11. Corporate Address	414 Nicollet Mall - 8th Floor Minneapolis, Minnesota 55401
12. Corporate Contact	G. H. Neils (Oper.) L. O. Mayer (Lic.)
13. Architect/Engineer	Pioneer Services & Eng. Co.
14. Nuclear Steam Supply System	Westinghouse
15. Constructor	Northern States Power Co.

Regulatory Information

16. IE Region responsible	III
17. IE Branch Chief	R. Heishman
18. IE Senior Inspector	R. F. Warnick
19. IE Resident Inspector	C. D. Feierabend 612-388-8209
20. Licensing Project Manager	M. Grotenhuis
21. Docket Number	50-306
22. License Number/Date of Issue	DPR-60 issued 10/29/74
23. Public Document Room	Environmental Conservation Library Minneapolis Public Library 300 Nicollet Mall Minneapolis, Minnesota 55401

INSPECTION STATUS

ENFORCEMENT STATUS

ENVIRONMENTAL TECHNICAL SPECIFICATION 2.4.3 REQUIRES THAT THE CONTENTS OF THE NEUTRALIZING TANK SHALL BE CONTROLLED SO THAT DISCHARGES FROM THE TANK SHALL HAVE A PH VALUE OF NOT LESS THAN 6.5 NOR GREATER THAN 8.5. CONTRARY TO THE ABOVE REQUIREMENT, ON OCTOBER 7, 1979, A NEUTRALIZING TANK BATCH WAS RELEASED WITH A PH OF 11.65. (7926 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

Routine normal operations after completion of refueling outage.

+LAST IE SITE INSPECTION DATE: January 15-18, 1980

+ INSPECTION REPORT NO: 80-03

SUMMARY

INSPECTION SUMMARY

+ Inspection on December 1-31, (79-24): Routine resident inspection of plant operations, maintenance, security, review and audit, calibration, radiation protection, radioactive waste system, followup of IE bulletins and circulars, preparation for Unit 2 refueling, inspection of new fuel for Unit 2, and core performance. The inspection involved 71 inspector hours onsite by the resident inspector. No items of noncompliance or deviations were identified.

+ Inspection on December 19-21, (79-26): Routine, announced inspection of confirmatory measurements for Unit 2, including the licensee's program for quality control of analytical measurements; evaluation of the licensee's analytical results of plant effluent samples obtained during a previous inspection; collection of plant effluent samples for future comparative analysis; and submission of a spiked particulate filter, charcoal adsorber and liquid sample for future comparative analyses; and environmental protection, including administrative and procedural control system; quality assurance and control in analytical measurements; implementation of the radiological environmental monitoring program (REMP); review of Environmental Event Reports (EER), and licensee actions on previously identified matters. Of the 10 areas inspected, no apparent items of noncompliance or deviations were identified in nine areas; one apparent item of noncompliance was identified in one area (neutralizing tank batch release with pH greater than 8.5).

+ Inspection on January 8-10, (80-02): Routine, unannounced inspection of the radiation protection program for operating and refueling, including: qualifications; licensee audits; training; radiation protection procedures; advance planning and preparation for refueling; instruments and equipment; exposure control; posting, labeling, and control; notification and reports; IE Bulletin 79-19; and surveys. The inspection involved 25 inspector hours onsite by one NRC inspector. Of the 11 areas inspected, no items of noncompliance or deviations were identified.

+ Inspection on January 15-18, (80-03): Inservice Inspection (ISI) program, implementing procedures, observation of work activities, and review and evaluation of nondestructive examinations (NDE) data and reports of ISI; review and evaluation of radiographic examination results related to IE Bulletin 79-13; review of data reports of (NDE) examination related to IE Bulletin 79-17. The inspection involved 21 inspector hours onsite by one NRC inspector. Of the six areas inspected, no apparent items of noncompliance were identified in five areas; one apparent item of noncompliance was identified in one area (failure to follow NDE procedures).

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

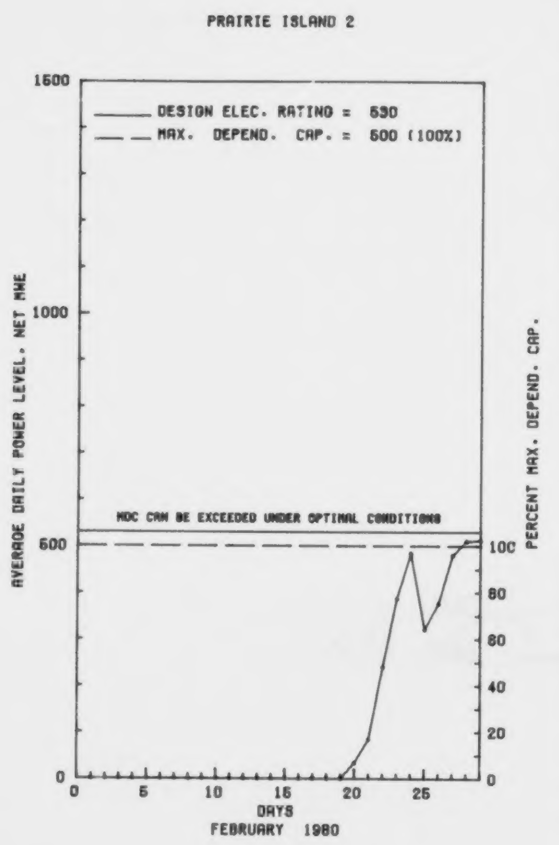
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
	NONE			

REVIEWED BY OMPA LOEB: T.C. Cintula **DATE:** 4·10·80

REVIEWED BY NRR DOR: W. J. Ross **DATE:** 4-11-80

PRAIRIE ISLAND 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-306
2. Utility Contact: A. A. Hunstad (612) 388-4767
3. Licensed Thermal Power (MWt): 1650
4. Nameplate Rating (Gross MWe): 659 x 0.9 = 593
5. Design Electrical Rating (Net MWe): 530
6. Maximum Dependable Capacity (Gross MWe): 531
7. Maximum Dependable Capacity (Net MWe): 500
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	696.0	1,440.0	45,526.0
12. Hours Reactor Was Critical	263.4	311.3	39,449.3
13. Reactor Reserve Shutdown Hours	.0	.0	1,406.0
14. Hours Generator On-Line	224.2	269.5	38,670.5
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	273,722	334,450	60,006,953
17. Gross Electrical Energy (MWH)	89,600	109,610	19,163,740
18. Net Electrical Energy (MWH)	78,936	95,397	17,934,029
19. Unit Service Factor	32.2	18.7	84.9
20. Unit Availability Factor	32.2	18.7	84.9
21. Unit Capacity Factor (MDC Net)	22.7	13.2	78.8
22. Unit Capacity Factor (DER Net)	21.4	12.5	74.3
23. Unit Forced Outage Rate	4.5	3.8	4.8

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	01/02/80	S	449.4	C	4				COMPLETION OF REFUELING OUTAGE.
	02/20/80	S	12.0	B	1				SHUTDOWN TO CONDUCT TURBINE OVERSPEED TEST AND GENERATOR SHORT-CIRCUIT TEST.
	02/21/80	F	7.9	B	1				SHUTDOWN TO REPAIR TURBINE STEAM LEAK.
	02/25/80	F	2.6	B	1				SHUTDOWN TO REPLACE PACKING IN FW REGULATING VALVE.

SUMMARY

THE UNIT RETURNED TO SERVICE ON 02/19/80 FROM A REFUELING OUTAGE. THE UNIT WAS TAKEN OFF LINE AGAIN ON 02/20/80 FOR TURBINE OVERSPEED AND GENERATOR SHORT-CIRCUIT TESTING; ON 02/21/80 TO REPAIR A TURBINE STEAM LEAK; AND ON 02/25/80 TO REPLACE PACKING IN A FEEDWATER REGULATING VALVE.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

QUAD CITIES 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1. Location (state/county) (nearest population center)	Illinois/Rock Island 20 miles NE of Moline
2. Electric Reliability Council	Main
3. Type of Reactor	BWR
4. Power Level (MWT)	2511
5. Design Electrical Rating (MWE Net)	789
6. Date of Initial Criticality	October 18, 1971
7. Date of Commercial Operation	February 18, 1973
8. Condenser Cooling Method	Spray Canal
9. Condenser Cooling Water Source	Mississippi River

Utility & Contractor Information

10. Licensee	Commonwealth Edison Co.
11. Corporate Address	P. O. Box 767 Chicago, Illinois 60690
12. Corporate Contact	D. L. Peoples
13. Architect/Engineer	Sargent and Lundy
14. Nuclear Steam Supply System	GE
15. Constructor	United Engineers & Constructors, Inc.

Regulatory Information

16. IE Region responsible	III
17. IE Branch Chief	R. Heishman
18. IE Senior Inspector	R. Spessard
19. IE Resident Inspector	N. J. Chrissotimus (309) 654-2227
20. Licensing Project Manager	R. B. Bevan
21. Docket Number	50-254
22. License Number/Date of Issue	DPR-29 issued 10/1/71
23. Public Document Room	Moline Public Library 504 17th Street Moline, Illinois 61265

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

FACILITY ITEMS

+ During the last two calendar quarters of 1979, the gaseous effluent release rate of radioiodine and radioactive particulates with half-lives greater than eight days has remained at or below the design objective for the site after exceeding twice the design objective release rate for each of the preceeding four calendar quarters. (79-30)

PLANT STATUS

Unit operating routinely.

+LAST IE SITE INSPECTION DATE: January 10, 1980

+ INSPECTION REPORT NO: 80-02

SUMMARY

INSPECTION SUMMARY

+ Inspection on December 11-14, (79-30): Nonroutine, announced inspection to review, since mid-year, the licensee's data, reports, and experience in meeting the design objective gaseous release rate for radioiodine and radioactive materials in particulate form with half-lives greater than eight days. The inspection involved 12 inspector hours onsite by one NRC inspector. No items of noncompliance or deviations were identified in the area reviewed.

+ Inspection on December 5, 1979 through January 14, 1980, (79-31): Investigation was conducted into the circumstances surrounding the closure of two core spray suction valves to the Quad-Cities Unit 1 Emergency Core Cooling System (ECCS). Investigation involved a review of procedures and records, interviews with plant personnel and discussions with the licensee and representatives of the Rock Island Office, Federal Bureau of Investigation (FBI). The investigation involved 80 inspector hours onsite by one inspector and one investigator. Investigation established on December 3, 1979 at 2400 hours, licensee discovered that two independently controlled core spray suction valves in the Unit 1 ECCS were found in a closed position. Interviews with plant personnel, and a review of maintenance and operations records indicated that there was no immediately discernable cause for the closure; both intentional and accidental manipulations of two valves were considered, but no positive conclusion was reached. No items of noncompliance were identified during the course of this investigation.

+ Inspection on January 10, (80-02): A special announced meeting was conducted to discuss iodine and particulate releases at Quad-Cities Nuclear Power Station; recent personnel errors at Quad-Cities Nuclear Power Station; Investigation of ECCS valve closure at Quad-Cities Nuclear Power Station; unplanned release of radioactive liquid effluent at Quad-Cities Nuclear Power Station. The meeting involved 12 inspector hours at the corporate office by six NRC personnel.

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
80-01/ 03L-0	01/11/80	02/01/80	30 Day LER	Differential pressure transmitter 1-8741-51 was found not to indicate properly.
80-02/ 03L-0	01/11/80	02/07/80	30 Day LER	SBGT charcoal absorber leakage in excess of limits.

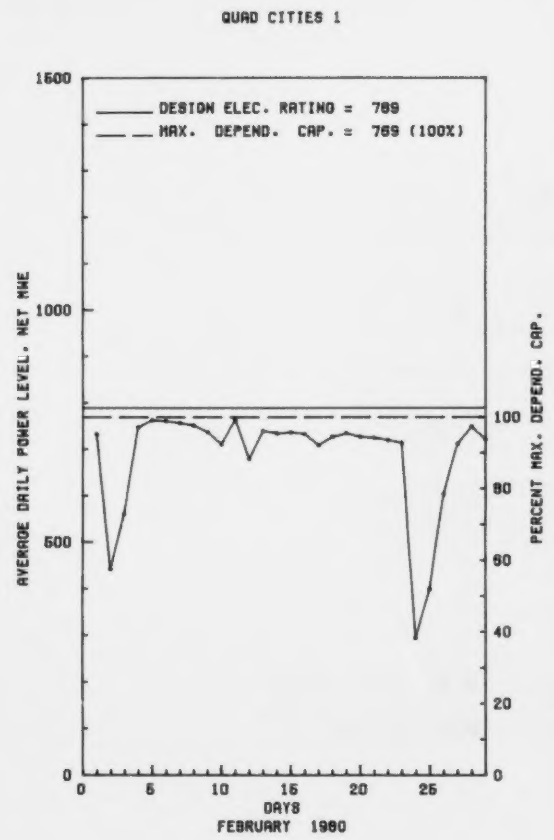
==

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 4-10-80

REVIEWED BY NRR DOR: R B Bevan **DATE:** 4/11/80

QUAD CITIES 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-254
2. Utility Contact: D. Hannum (309) 654-2241
3. Licensed Thermal Power (MWt): 2511
4. Nameplate Rating (Gross MWe): 920 x 0.9 = 828
5. Design Electrical Rating (Net MWe): 789
6. Maximum Dependable Capacity (Gross MWe): 813
7. Maximum Dependable Capacity (Net MWe): 769
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	696.0	1,440.0	68,400.0
12. Hours Reactor Was Critical	696.0	1,440.0	56,154.1
13. Reactor Reserve Shutdown Hours	.0	.0	3,421.9
14. Hours Generator On-Line	696.0	1,440.0	53,480.9
15. Unit Reserve Shutdown Hours	.0	.0	909.2
16. Gross Thermal Energy (MWH)	1,532,337	3,279,550	107,823,531
17. Gross Electrical Energy (MWH)	500,761	1,076,887	34,699,941
18. Net Electrical Energy (MWH)	477,145	1,026,916	32,442,452
19. Unit Service Factor	100.0	100.0	78.2
20. Unit Availability Factor	100.0	100.0	79.5
21. Unit Capacity Factor (MDC Net)	89.1	92.7	61.7
22. Unit Capacity Factor (DER Net)	86.9	90.4	60.1
23. Unit Forced Outage Rate	.0	.0	7.6

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
2	02/02/80	F	.0	B	5				LOAD REDUCTION FOR INVESTIGATION OF POTENTIAL CONDENSER TUBE LEAKS.
3	02/24/80	S	.0	H	5				LOAD REDUCTION FOR ONE-LINE CONTROL ROD SEQUENCE EXCHANGE.

SUMMARY

OPERATION WAS ROUTINE DURING THE REPORT PERIOD, WITH LOAD REDUCTIONS ON 02/02/80 TO INVESTIGATE POTENTIAL CONDENSER TUBE LEAKS, AND ON 02/24/80 FOR CONTROL ROD SEQUENCE EXCHANGE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

QUAD CITIES 2

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Illinois/Rock Island 20 miles NE of Moline
2.	Electric Reliability Council	Main
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2511
5.	Design Electrical Rating (MWE Net)	789
6.	Date of Initial Criticality	April 26, 1972
7.	Date of Commercial Operation	March 10, 1973
8.	Condenser Cooling Method	Spray Canal
9.	Condenser Cooling Water Source	Mississippi River

Utility & Contractor Information

10.	Licensee	Commonwealth Edison Co.
11.	Corporate Address	P. O. Box 767 Chicago, Illinois 60690
12.	Corporate Contact	D. L. Peoples
13.	Architect/Engineer	Sargent & Lundy
14.	Nuclear Steam Supply System	GE
15.	Constructor	United Engineers & Constructors, Inc.

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Senior Inspector	R. Spessard
19.	IE Resident Inspector	N. J. Chrissotimus (309) 654-2227
20.	Licensing Project Manager	R. B. Bevan
21.	Docket Number	50-265
22.	License Number/Date of Issue	DPR-30 issued 3/21/72
23.	Public Document Room	Moline Public Library 504 17th Street Moline, Illinois 61265

INSPECTION STATUS

ENFORCEMENT STATUS

TECHNICAL SPECIFICATION 3.8.D.1 STATES, "RADIOACTIVE LIQUIDS RELEASED FROM THE FACILITY SHALL BE CONTINUOUSLY MONITORED. TO ACCOMPLISH THIS, EITHER THE RADIATION MONITOR ON THE DISCHARGE LINE, OR THE DISCHARGE BAY SAMPLER SHALL BE OPERABLE OR GRAB SAMPLES SHALL BE TAKEN IN THE DISCHARGE BAY DURING THE COURSE OF THE DISCHARGE." CONTRARY TO THIS REQUIREMENT, THE UNPLANNED RADIOACTIVE LIQUID RELEASE WHICH BEGAN ABOUT 10:00 A.M. ON DECEMBER 9, 1979, WHILE DRAINING THE 2B RHR HEAT EXCHANGER VIA THE INSTRUMENT MANIFOLD WAS NOT CONTINUOUSLY MONITORED. (7927 INFRACTION)

TECHNICAL SPECIFICATION 6.2.A REQUIRES THAT DETAILED WRITTEN PROCEDURES INCLUDING APPLICABLE CHECKOFF LISTS BE PREPARED, APPROVED AND ADHERED TO FOR PREVENTIVE AND CORRECTIVE MAINTENANCE OPERATIONS WHICH COULD HAVE AN EFFECT ON THE SAFETY OF THE FACILITY. CONTRARY TO THE ABOVE, ON DECEMBER 9, 1979 TEMPORARY PROCEDURE 1261, "DRAINING UNIT 2 RHR SERVICE WATER PIPING," WAS NOT ADHERED TO IN THREE INSTANCES AS DELINEATED BELOW, RESULTING IN AN UNPLANNED RADIOACTIVE LIQUID EFFLUENT RELEASE TO THE MISSISSIPPI RIVER AS WELL AS RADIOACTIVELY CONTAMINATING A SMALL PORTION OF THE SITE GROUNDS AND THE STORM DRAINAGE SYSTEM. (A) THE PROCEDURE REQUIRED A HOSE BE CONNECTED TO ONE SPECIFIED INSTRUMENT LINE, WHEREAS THE HOSE WAS CONNECTED TO THE INSTRUMENT MANIFOLD; (B) THE PROCEDURE REQUIRED DRAINING THE SERVICE WATER SIDE OF THE RHR HEAT EXCHANGER BY OPENING ONE SPECIFIED INSTRUMENT LINE, WHEREAS, ALL INSTRUMENT LINES CONNECTED TO THE MANIFOLD WERE OPENED; AND (C) THE PROCEDURE REQUIRED A HOSE BE EXTENDED FROM THE OUTSIDE OF THE REACTOR BUILDING TO THE STORM SEWER, WHEREAS, NO SUCH CONNECTION WAS MADE. (7927 INFRACTION)

TECHNICAL SPECIFICATION 6.2.B REQUIRES THAT RADIATION CONTROL PROCEDURES BE MAINTAINED AND ADHERED TO. RADIATION CONTROL PROCEDURE QRP 100-1 TITLED, "RADIATION CONTROL STANDARDS" STATES IN PARAGRAPH F.16.A.2., "EXCEPT AS NOTED, A SPECIAL WORK PERMIT (SWP) WILL BE REQUIRED WHEN PERSONNEL MAY, OR ARE EXPECTED TO, EXCEED A DAILY WHOLE BODY DOSE OF 50 MILLIREMS. IF, HOWEVER, A RADIATION PROTECTION MAN IS IN CONTINUOUS ATTENDANCE AT THE JOB SITE WHILE A JOB IS IN PROGRESS, HE MAY AUTHORIZE A DAILY WHOLE BODY DOSE OF UP TO 100 MILLIREM PER DAY WITHOUT REQUIRING AN SWP." CONTRARY TO THE ABOVE, TWO WORKERS EACH RECEIVED GREATER THAN 50 MREM DOSE (75 MREM AND 130 MREM) ON DECEMBER 9, 1979, WHILE PERFORMING WORK ASSIGNMENTS AT THE INSTRUMENT MANIFOLD NEAR THE 2B RHR HEAT EXCHANGER AND OTHER LOCATIONS WITHOUT AUTHORIZATION PROVIDED BY AN SWP OR CONTINUOUS H.P. SURVEILLANCE. (7927 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

FACILITY ITEMS

+ During the last two calendar quarters of 1979, the gaseous effluent release rate of radioiodine and radioactive particulates with half-lives greater than eight days has remained at or below the design objective for the site after exceeding twice the design objective release rate for each of the preceeding four calendar quarters. (79-28)

PLANT STATUS

+ The unit was shut down on November 25, 1979 to begin a scheduled 13-week refueling and maintenance outage. The outage work is nearing completion, and the unit is scheduled for return to service about March 15, 1980.

+LAST IE SITE INSPECTION DATE: January 9-11, 1980

+ INSPECTION REPORT NO: 80-03

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
80-01/ 03L-0	01/08/80	02/07/80	30 Day LER	SBLC relief valve actuated in excess of setpoint.
80-02/ 03L-0	01/08/80	02/07/80	30 Day LER	Air leakage of RHR service water pump vault electrical penetration.

SUMMARY

INSPECTION SUMMARY

+ Inspection on December 11-14, (79-27): Special, inspection of unplanned radiological effluent release. The inspection involved 39 inspector hours onsite by two NRC inspectors. Three items of noncompliance (failure to follow a temporary procedure; failure to follow a radiation control procedure; and failure to continuously monitor an unplanned radioactive liquid release from the facility) were identified.

+ Inspection on December 11-14, (79-28): Nonroutine, announced inspection to review, since mid-year, the licensee's data, reports, and experience in meeting the design objective gaseous release rate for radioiodine and radioactive materials in particulate form with half-lives greater than eight days. The inspection involved 12 inspector hours onsite by one NRC inspector. No items of noncompliance or deviations were identified in the area reviewed.

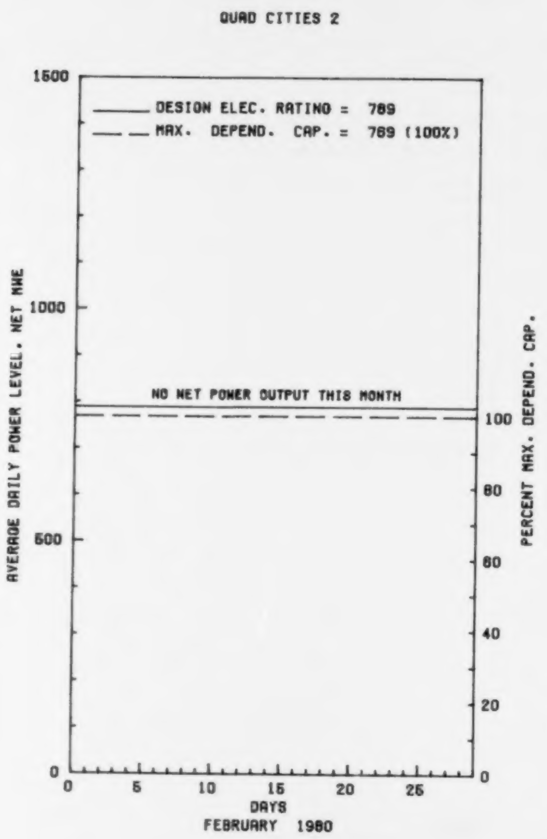
+ Inspection on January 9-11, (80-03): Facility Organizations and Operation, Measurement and Control, Inventory, Shipping and Receiving, Records and Reports, Internal Controls and Management of Material Control System. The inspection involved 38 inspector hours onsite by two NRC inspectors and was begun during the regular hours. The licensee was found to be in compliance with NRC requirements in the seven areas examined during the inspections except for the following one item: licensee failed to prepare NRC Form 741 for transfer of SNM between lcoations assigned different Reporting Identification Symbols (RIS).

+ Inspection on January 18, (80-04): A special announced meeting was conducted to discuss iodine and particulate releases at Quad-Cities Nuclear Power Station; recent personnel errors at Quad-Cities Nuclear Power Station; investigation of ECCS valve closure at Quad-Cities Nuclear Power Station; unplanned release of radioactive liquid effluent at Quad-Cities Nuclear Power Station. The meeting involved 12 inspector hours at the corporate office by six NRC personnel.

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 4-10-80

REVIEWED BY NRR DOR: RBBevan **DATE:** 4/11/80

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-265
2. Utility Contact: D. Mannum (309) 654-2241
3. Licensed Thermal Power (MWt): 2511
4. Nameplate Rating (Gross MWe): 920 x 0.9 = 828
5. Design Electrical Rating (Net MWe): 789
6. Maximum Dependable Capacity (Gross MWe): 813
7. Maximum Dependable Capacity (Net MWe): 769
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	696.0	1,440.0	67,510.0
12. Hours Reactor Was Critical	.0	.0	53,128.9
13. Reactor Reserve Shutdown Hours	.0	.0	2,985.8
14. Hours Generator On-Line	.0	.0	50,791.8
15. Unit Reserve Shutdown Hours	.0	.0	702.9
16. Gross Thermal Energy (MWH)	0	0	103,016,109
17. Gross Electrical Energy (MWH)	20	27	32,863,077
18. Net Electrical Energy (MWH)	-13,270	-27,085	30,715,440
19. Unit Service Factor	.0	.0	75.2
20. Unit Availability Factor	.0	.0	76.3
21. Unit Capacity Factor (MDC Net)	.0	.0	59.2
22. Unit Capacity Factor (DER Net)	.0	.0	57.7
23. Unit Forced Outage Rate	.0	.0	9.3

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 03/15/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
13	11/25/79	S	696.0	C	4				UNIT TWO CYCLE 4 REFUELING OUTAGE.

SUMMARY

THE UNIT REMAINED SHUTDOWN THROUGHOUT THE REPORT PERIOD FOR REFUELING.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

RANCHO SECO

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	California/Sacramento 25 miles SE of Sacramento
2.	Electric Reliability Council	WSCC
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2772
5.	Design Electrical Rating (MWE Net)	918
6.	Date of Initial Criticality	September 16, 1974
7.	Date of Commercial Operation	April 17, 1975 (92.6% of full power)
8.	Condenser Cooling Method	Closed Cycle
9.	Condenser Cooling Water Source	Natural Draft Cooling Towers - Folsom Canal

Utility & Contractor Information

10.	Licensee	Sacramento Municipal Utility District
11.	Corporate Address	6201 South Street P. O. Box 15830 Sacramento, California
12.	Corporate Contact	J. J. Mattimoe Assistant General Manager, Chief Engineer
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	B&W
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	V
17.	IE Branch Chief	J. L. Crews
18.	IE Senior Inspector	B. H. Faulkenberry
19.	IE Resident Inspector	H. L. Canter (209) 748-2791
20.	Licensing Project Manager	D. J. Garner
21.	Docket Number	50-312
22.	License Number/Date of Issue	DPR-54, 8/16/74
23.	Public Document Room	Business and Municipal Department Sacramento City-County Library 828 I Street Sacramento, California 95814

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO 10 CFR 50, APPENDIX B, CRITERION V, ANSI N.18.7, AS INVOKED BY APPENDIX A, SECTION 6.8 OF DRR-54, WHICH REQUIRES INSTRUCTIONS SHALL INCLUDE APPROPRIATE ACCEPTANCE CRITERIA, ON JULY 25, 1979 THE LICENSEE DID NOT SPECIFY ACCEPTANCE CRITERIA NOR WAS ACCEPTANCE CRITERIA AVAILABLE FOR DETERMINING THE ACCEPTABILITY OF MEASURED CONCRETE EXPANSION ANCHOR BOLT LENGTHS. THE DATA SHEETS HAD BEEN SIGNED BY THE ENGINEER AND QC.
(7917 INFRACTION)

CONTRARY TO TECHNICAL SPECIFICATION 2.2, APPENDIX B, THE TOTAL CHLORINE CONCENTRATION IN PLANT EFFLUENT EXCEEDED THE LIMIT OF 0.2 MG/LITER ON OCTOBER 24, NOVEMBER 20, DECEMBER 3, DECEMBER 5 AND DECEMBER 14, 1979.
(8002 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST (CITE RO REPORT NUMBER, IF APP.)

SYSTEMS AND COMPONENT PROBLEMS:

None.

FACILITY ITEMS (PLANS AND PROCEDURES):

None.

MANAGERIAL ITEMS:

None.

PLANT STATUS:

Plant shutdown for refueling.

LAST IE SITE INSPECTION DATE: 02/01-29/80+

INSPECTION REPORT NO. 50-312/80-06+

SUMMARY

INSPECTION SUMMARY

+ Inspection on December 13, 1979 (Report No. 50-312/79-29)
Areas Inspected: This inspection involved the examination of a shipment of low level radioactive waste for disposal. The shipment was inspected for compliance with NRC and DOT regulations. This inspection required one hour on site by one NRC inspector.

Results: No items of noncompliance were identified relative to NRC and DOT regulations.

+ Inspection on January 18-25, 1980 (Report No. 50-312/80-01)
Report being prepared; to be reported next month.

+ Inspection on January 21-25, 1980 (Report No. 50-312/80-02)
Areas Inspected: Refueling preparations including radiation safety procedures; radiation safety training; personnel monitoring and exposure control; respiratory protection program; instrument calibration; surveys; followup of reported unusual events; licensee action on IE Bulletin 79-19; allegation of possible problems with shipments of liquid radioactive waste; and a tour of the facility. The inspection involved 37 hours on site by one NRC inspector.

Results: Of the ten areas inspected, no items of noncompliance or deviations were identified in nine areas. One item of noncompliance was found in one area (Deficiency - Total chlorine concentration in plant effluent exceeded limit).

+ Inspection on January 2-31, 1980 (Report No. 50-312/80-03)
Areas Inspected: Routine inspections of plant operations; physical security; implementation of strike plan; refueling preparations; LER followup and reporting; followup on inspector identified items; and independent inspection effort. The inspection involved 114 inspector hours by the NRC Resident Inspectors and 24 inspector hours by one NRC Region 5 based inspector.

Results: No items of noncompliance or deviations were identified in six areas. One item of noncompliance was identified in one area (LER followup and reporting).

NRC POWER RESTRICTIONS: None

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-24-L	01-09-80	01-25-80	30 Day LER	NSRW not available to MU pump when in standby for HPI pump.
80-03-L	01-09-80	02-06-80	30 Day LER	SF valve 23508 (MU TK isol) without power for 42 hours.
80-04-L	01-10-80	02-11-80	30 Day LER	RCS leakage exceeded TS limit of 1 gpm-leaking gland on spray valve.
80-05-L	01-17-80	02-14-80	30 Day LER	Leakage from DHR pump (P-216A) exceeded TS limit of .63 GPH.
80-06-T	02-04-80	02-04-80	14 Day LER	Bolt in anchor valves may be incompatible with valve components.
80-06-T	02-04-80	02-18-80	14 Day LER	Stud failure in anchor valve.

+ Inspection on February 12-15, 1980 (Report No. 50-312/80-04)
Report being prepared; to be reported next month.

+ Inspection on January 30, February 1, 12-15 and 17-18, 1980 (Report No. 50-312/80-05)
Areas Inspected: Routine, unannounced inspection by Regional based inspector of the licensee's activities performed in response to IE Bulletin 79-02, "Pipe Support Base Plate Design Using Concrete Expansion Anchor Bolts" and of licensee's activities performed for the inservice inspection for the 1980 refueling outage. The inspection involved 67 inspector hours on site by one NRC inspector.

Results: No items of noncompliance or deviations were identified.

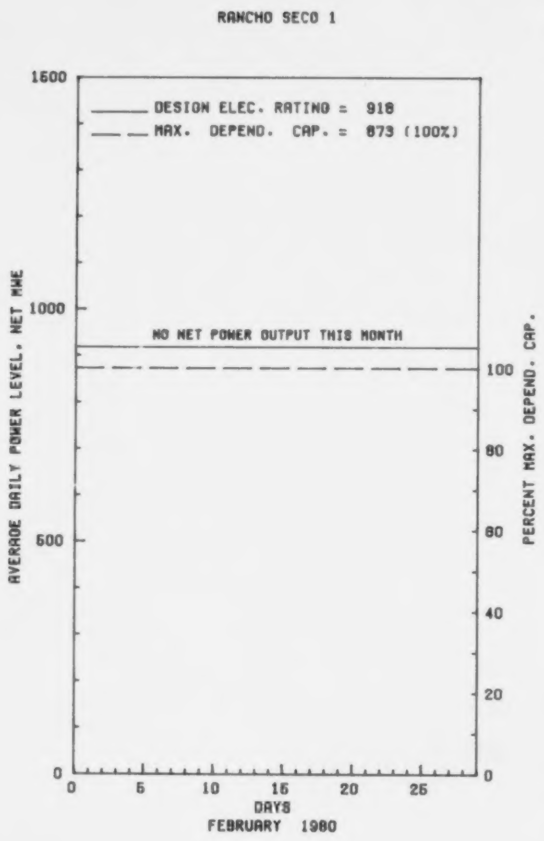
+ Inspection on February 1-29, 1980 (Report No. 50-312/80-06)
Report being prepared; to be reported next month.

REVIEWED BY OMPA LOEB: E Boke **DATE:** 4/10/80

REVIEWED BY NRR DOR: **DATE:** [illegible]

RANCHO SECO

AVERAGE DAILY POWEF LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-312
2. Utility Contact: R. W. Colombo (916) 452-3211
3. Licensed Thermal Power (MWt): 2772
4. Nameplate Rating (Gross MWe): 1070 x 0.9 = 963
5. Design Electrical Rating (Net MWe): 918
6. Maximum Dependable Capacity (Gross MWe): 917
7. Maximum Dependable Capacity (Net MWe): 873
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	696.0	1,440.0	42,697.0
12. Hours Reactor Was Critical	.0	271.0	26,344.5
13. Reactor Reserve Shutdown Hours	.0	.0	3,975.1
14. Hours Generator On-Line	.0	266.9	26,208.2
15. Unit Reserve Shutdown Hours	.0	.0	1,210.2
16. Gross Thermal Energy (MWH)	0	689,820	64,201,951
17. Gross Electrical Energy (MWH)	0	221,784	21,506,320
18. Net Electrical Energy (MWH)	0	206,442	20,317,332
19. Unit Service Factor	.0	18.5	59.0
20. Unit Availability Factor	.0	18.5	61.9
21. Unit Capacity Factor (MDC Net)	.0	16.4	54.5
22. Unit Capacity Factor (DER Net)	.0	15.6	51.8
23. Unit Forced Outage Rate	.0	14.5	32.0

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 04/01/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
02	01/14/80	S	696.0	C	4				3RD REFUELING OUTAGE CONTINUES.

SUMMARY

THE UNIT REMAINED SHUTDOWN THROUGHOUT THE REPORT PERIOD FOR REFUELING.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

ROBINSON 2

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	South Carolina/Darlington 5miles NW of Hartsville
2.	Electric Reliability Council	Southeastern
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2300
5.	Design Electrical Rating (MWE Net)	700
6.	Date of Initial Criticality	September 20, 1970
7.	Date of Commercial Operation	March 7, 1971
8.	Condenser Cooling Method	Recirculation from impounded lake
9.	Condenser Cooling Water Source	Robinson Impoundment

Utility & Contractor Information

10.	Licensee	Carolina Power and Light Company
11.	Corporate Address	411 Fayetteville Street Raleigh, North Carolina
12.	Corporate Contact	J. A. Jones Executive Vice President
13.	Architect/Engineer	Ebasco
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Ebasco

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R.C. Lewis
18.	IE Senior Inspector	P. J. Kellogg
19.	IE Principal Inspector	T. McHenry
20.	Licensing Project Manager	D. Neighbors
21.	Docket Number	50-261
22.	License Number/Date of Issue	DPR-23, 7/31/70 (5 Mwt), Full power authorization issued 9/23/70
23.	Public Document Room	Hartsville Memorial Library Home and Fifth Avenue Hartsville, South Carolina 29550

R-4

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

UNIT IS IN NORMAL OPERATION.

+ LAST IE SITE INSPECTION DATE: FEBRUARY 4-6, 1980

+ INSPECTION REPORT NO: 50-261/80-03

SUMMARY

INSPECTION SUMMARY

+ INSPECTION FEBRUARY 4-7 (80-01): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 34 INSPECTOR-HOURS ON SITE IN THE AREAS OF QUALITY CONTROL AND CONFIRMATORY MEASUREMENTS: INCLUDING REVIEW OF THE LABORATORY QUALITY CONTROL PROGRAM: AUDIT RESULTS; REVIEW OF CHEMICAL AND RADIOCHEMICAL PROCEDURES; AND COMPARISON OF THE ANALYSES OF SPLIT SAMPLES ANALYZED BY THE LICENSEE AND THE NRC REGION II MOBILE LABORATORY. OF THE FOUR AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN THREE AREAS; ONE APPARENT ITEM OF NONCOMPLIANCE WAS FOUND IN ONE AREA (INFRACTION - FAILURE TO HAVE AND FOLLOW WRITTEN APPROVED PROCEDURES FOR RADIOCHEMICAL MEASUREMENTS).

INSPECTION JANUARY 23-25 (80-02): INCLUDED REVIEW OF SECURITY ORGANIZATION; PROTECTED AREA BARRIER; VITAL AREA ALARMS AND ALARM STATIONS; ACCESS CONTROLS; DETECTION AIDS; AND ASSESSMENT AIDS. THE INSPECTION INVOLVED 30 INSPECTOR-HOURS ON SITE BY TWO NRC INSPECTORS. THE LICENSEE WAS FOUND TO BE IN COMPLIANCE WITH NRC REQUIREMENTS WITHIN THE SIX AREAS EXAMINED DURING THE INSPECTION EXCEPT FOR THE FOLLOWING TWO ITEMS: (1) FAILURE TO SEARCH VISITORS; AND (2) FAILURE TO MAINTAIN INTEGRITY OF THE PROTECTED AREA BARRIER.

INSPECTION FEBRUARY 4-6 (80-03): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 34 INSPECTOR-HOURS ON SITE IN THE AREAS OF RADIATION PROTECTION INCLUDING GENERAL EMPLOYEE TRAINING AND POSTING, LABELING AND CONTROL OF RADIOACTIVE MATERIALS; REVIEW OF LICENSEE ACTION ON IE BULLETIN 79-19; AND FOLLOWUP ON PREVIOUSLY IDENTIFIED ITEMS. OF THE FOUR AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN TWO AREAS; TWO ITEMS OF NONCOMPLIANCE WERE FOUND IN TWO AREAS (DEFICIENCY - FAILURE TO PROPERLY LABEL CONTAINERS OF RADIOACTIVE MATERIAL; AND INFRACTION - FAILURE TO FOLLOW PROCEDURES).

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

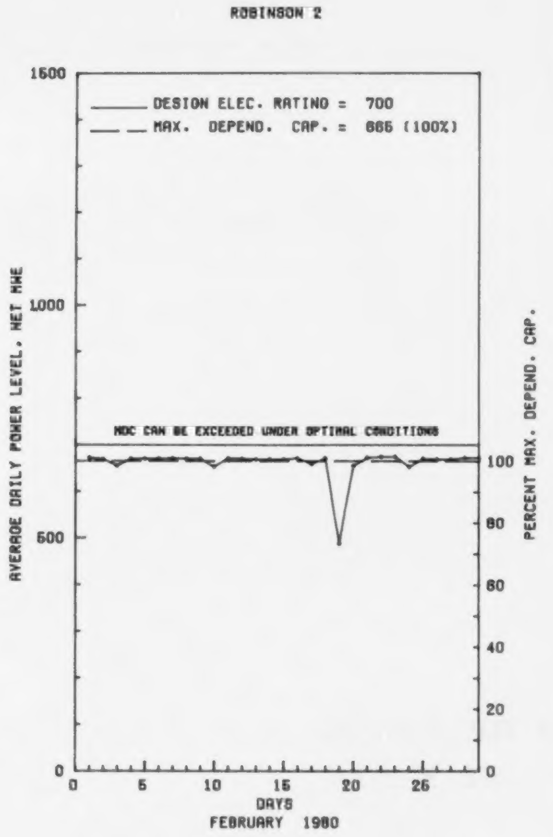
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
NONE				

REVIEWED BY OMPA LOEB: T. C. CINTULA **DATE:** 4·10·80

REVIEWED BY NRR DOR: Don Neighbors **DATE:** 4/11/80

ROBINSON 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-261
2. Utility Contact: M. L. Watford (803) 383-4524
3. Licensed Thermal Power (MWt): 2300
4. Nameplate Rating (Gross MWe): 854 x 0.9 = 769
5. Design Electrical Rating (Net MWe): 700
6. Maximum Dependable Capacity (Gross MWe): 700
7. Maximum Dependable Capacity (Net MWe): 665
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): 2200 MWt
10. Reasons for Restrictions, If Any:
EXCESSIVE MOISTURE CARRY-OVER TO H.P. TURBINE.

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	696.0	1,440.0	78,822.0
12. Hours Reactor Was Critical	696.0	1,440.0	61,646.9
13. Reactor Reserve Shutdown Hours	.0	.0	719.3
14. Hours Generator On-Line	696.0	1,440.0	60,130.9
15. Unit Reserve Shutdown Hours	.0	.0	23.2
16. Gross Thermal Energy (MWH)	1,475,827	3,069,782	123,620,11
17. Gross Electrical Energy (MWH)	483,612	1,010,370	39,946,883
18. Net Electrical Energy (MWH)	460,529	962,613	37,867,715
19. Unit Service Factor	100.0	100.0	76.3
20. Unit Availability Factor	100.0	100.0	76.3
21. Unit Capacity Factor (MDC Net)	99.5	100.5	72.2
22. Unit Capacity Factor (DER Net)	94.5	95.5	68.6
23. Unit Forced Outage Rate	.0	.0	12.9

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
REFUELING/MAINTENANCE, MAY 1980, 6 WEEKS
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
02-01	02/19/80	F	.0	B	5		HC	HTEXCH	POWER REDUCTION TO PLUG LEAKING MAIN CONDENSER TUBES. 10 TUBES WERE PLUGGED AND THE UNIT ESCALATED TO NORMAL OPERATING POWER LEVEL.

SUMMARY

OPERATION WAS ROUTINE AT A 2200 MWt LIMIT BECAUSE OF EXCESSIVE MOISTURE CARRYOVER TO THE HIGH PRESSURE TURBINE. LOAD WAS REDUCED BRIEFLY ON 02/19/80 TO PLUG LEAKING MAIN CONDENSER TUBES.

LEGEND

[1] Type:
F-Forced
S-Scheduled

[2] Reason:
A-Equipment Failure
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error
H-Other

[3] Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Continuations
5-Load Reduction
9-Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

SALEM 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	New Jersey/Salem 20 miles S of Wilmington, Del.
2.	Electric Reliability Council	MAAC
3.	Type of Reactor	PWR
4.	Power Level (MWT)	3338
5.	Design Electrical Rating (MWE Net)	1090
6.	Date of Initial Criticality	December 11, 1976
7.	Date of Commercial Operation	June 30, 1977
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Delaware River

Utility & Contractor Information

10.	Licensee	Public Service Electric & Gas Co.
11.	Corporate Address	80 Park Place Newark, New Jersey 07101
12.	Corporate Contact	F. P. Librizzi General Manager - Electric Prod.
13.	Architect/Engineer	PSE&G
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	UE&C

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	IE Senior Inspector	R. R. Keimig
19.	IE Resident Inspector	L. J. Norrholm (609-935-3850) (FTS 341-3000)
20.	Licensing Project Manager	W. Ross
21.	Docket Number	50-272
22.	License Number/Date of Issue	DPR-70, 8/13/76
23.	Public Document Room	Salem Free Public Library 112 West Broadway Salem, New Jersey 08079

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

- Leakage from the spent fuel pool liner of between 500-1000 cc/hr. Repairs are in progress on the suspected leak area.

PLANT STATUS:

Routine normal operations.

- LAST IE SITE INSPECTION DATE: 3/12/80
- INSPECTION REPORT NO: 50-272/80-08

SUMMARY

INSPECTION SUMMARY

- 50-272/79-32 - Nov 18 - Dec 15, 1979 & Jan 9, 1980: Routine inspections by the resident inspector of plant operations including: tours of the facility; log and record reviews; review of licensee events; IE Bulletins and Circulars; implementation of licensing commitments; and, followup on previous inspection items. The inspections involved 49 hours by the NRC resident inspector and 6 hours by one region-based inspector. Noncompliances: One (failure to follow procedures).
- 50-272/79-33 - Dec 16, 1979 - Feb 2, 1980: Routine inspections by the NRC resident inspector (40 hours) of plant operations including: tours of the facility; log and record reviews; review of licensee events; IE Bulletins; Immediate Action Letter followup; observations in the control room; and followup on previous inspection items. Noncompliances: None. Deviation: One (change made to the facility as described in the FSAR).
- 50-272/80-01 - Jan 22-25: Routine, unannounced inspection by a region-based inspector (29 hours) of licensee action on previous inspections findings; administrative control of safety related calibrations; surveillance calibrations of safety related components and equipment required by Technical Specifications; calibrations required by Technical Specifications of components and equipment association with safety related systems and/or functions; calibration and control of test equipment; witnessing of calibrations; technician qualifications; and facility tour. Noncompliances: One (failure to properly review calibration data).

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

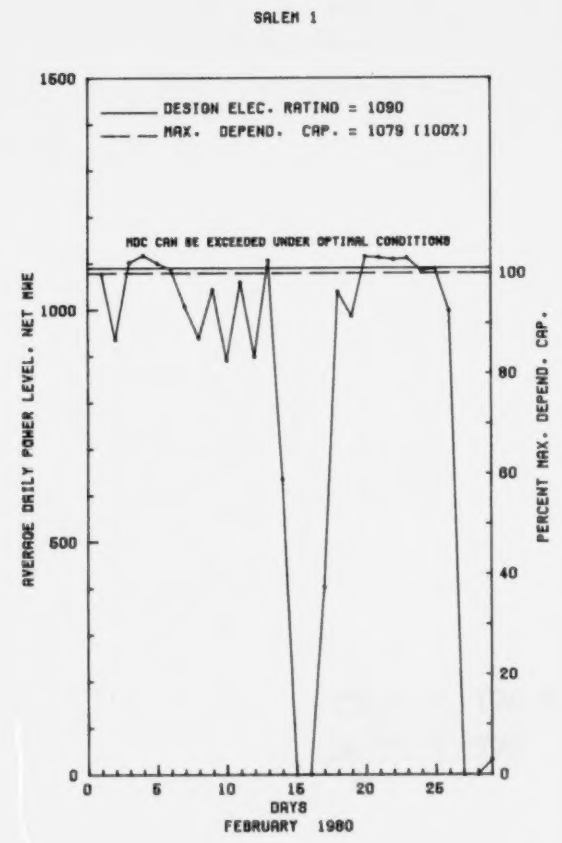
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-073/ 04L	12/07/79	01/03/80	30 Day LER	Failure to continuously monitor liquid release from #11 monitor tank due to failed liquid release monitor channel
79-074/ 03L	12/25/79	01/24/80	30 Day LER	During startup operations the containment sump level monitoring system was declared inoperable due to a failure of the alarm typewriter circuit
79-076/ 03L	12/20/79	01/15/80	30 Day LER	Due to personnel error #12 CVCS holdup tank was overpressurized resulting in deformation to the tank
79-077/ 03L	12/24/79	02/22/80	30 Day LER	During startup #11 ASD breaker tripped due to fault in the logic circuit causing 1A set to be inoperable
79-078/ 03L	12/26/79	01/24/80	30 Day LER	Due to inproper closing sequence of the high speed breakers the #12 containment fan would not start in high speed
79-079/ 03L	12/30/79	01/31/80	30 Day LER	Failure of the level indicating circuit on the RWST resulted in a level decrease below technical specification limits
79-080/ 03L	12/27/79	01/31/80	30 Day LER	During normal startup, the 1E 4KV bus was lost, which resulted in loss of one RCP
79-081/ 04L	08/23/79	12/18/79	Environmental Deviation	During normal river trawling an Atlantic Loggerhead Turtle was captured and released unharmed. The Loggerhead Turtle in on the Endangered Species List
80-001/ 03L	01/01/80	01/24/80	30 Day LER	Due to failure of a signal isolator the target band for axial flux difference was exceeded
80-002/ 03L	01/17/80	02/05/80	30 Day LER	Failure to perform a functional test of power range low trip setpoint when greater than 25 percent power
80-004/ 03L	01/14/80	02/14/80	30 Day LER	With the plant in hot standby a low recirc flow indication was received on the boron injection tank. Cause was attributed to a clogged flow meter
80-005/ 03L	01/17/80	02/11/80	30 Day LER	During power operations containment pressure rose above TS limits
80-006/ 03L	01/23/80	02/21/80	30 Day LER	The meterological tower instrumentation was inoperable momentarily due to a power failure
80-007/ 01T	01/28/80	02/11/80	24 Hours	A check valve presently installed in the demineralized water supply to the auxiliary feedwater storage tanks is not in conformance with design requirements
80-008/ 03L	01/29/80	02/15/80	30 Day LER	During maintenance activities performed on 1VC6 containment pressure vacuum relief valve it was discovered that replacement parts were not qualified
80-009/ 03L	01/17/80	02/14/80	30 Day LER	During a routine inspection of the spent fuel pit, a small leak (approximately 1.9 gph) was discovered at a tell tale drain

REVIEWED BY OMPA LOEB: T.C. Cintula **DATE:** 4·10·80

REVIEWED BY NRR DOR: W.J. Ross **DATE:** 4-11-80

SALEM 1

AVERAGE DAILY POWER LEVEL(MWe)



OPERATING STATUS

1. Docket Number: 50-272
2. Utility Contact: L. K. Miller (609) 365-7000
3. Licensed Thermal Power (MWt): 3338
4. Nameplate Rating (Gross MWe): 1300 x 0.9 = 1170
5. Design Electrical Rating (Net MWe): 1090
6. Maximum Dependable Capacity (Gross MWe): 1124
7. Maximum Dependable Capacity (Net MWe): 1079
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	696.0	1,440.0	23,401.0
12. Hours Reactor Was Critical	653.2	1,349.5	11,437.8
13. Reactor Reserve Shutdown Hours	.0	.0	22.7
14. Hours Generator On-Line	568.6	1,250.3	10,781.4
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,780,714	3,570,430	31,106,726
17. Gross Electrical Energy (MWH)	602,650	1,188,730	10,356,230
18. Net Electrical Energy (MWH)	574,704	1,130,986	9,760,576
19. Unit Service Factor	81.7	86.8	46.1
20. Unit Availability Factor	81.7	86.8	46.1
21. Unit Capacity Factor (MDC Net)	76.5	72.8	38.7
22. Unit Capacity Factor (DER Net)	75.8	72.1	38.3
23. Unit Forced Outage Rate	18.3	13.2	42.4

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-082	02/02/80	F	.0	A	5		HH	FILTER	CLEAN COND. PUMP SUCTION STRAINERS.
80-034	02/07/80	F	.0	A	5		HH	FILTER	CLEAN COND. PUMP SUCTION STRAINERS.
80-035	02/08/80	F	.0	A	5		HH	FILTER	BACKFLUSH 11, 12 HEATER DRAIN PUMPS.
80-039	02/08/80	F	.0	A	5		HH	FILTER	CLEAN COND. PUMP SUCTION STRAINERS.
80-041	02/10/80	F	.0	A	5		HH	FILTER	CLEAN COND. PUMP SUCTION STRAINERS.
80-043	02/10/80	F	.0	A	5		HH	FILTER	BACKFLUSH 11, 12 HEATER DRAIN PUMPS.
80-045	02/11/80	F	.0	A	5		HH	FILTER	CLEAN COND. PUMP SUCTION STRAINERS.
80-046	02/11/80	F	.0	A	5		HH	FILTER	CLEAN 11, 12 HEATER DRAIN PUMP SUCTION STRAINERS.
80-047	02/12/80	F	.0	A	5		HH	FILTER	CLEAN COND. PUMP SUCTION STRAINERS.
80-049	02/14/80	F	60.0	A	3		HB	MECFUN	#14 STEAM GENERATOR LO-LEVEL, LO-FLOW DUE TO 14BF19 VALVE POSITIONER FAILURE.
80-055	02/19/80	F	.0	A	5		HF	FILTER	CLEAN 13A CIRC. WATER PUMP WATERBOX.
80-060	02/26/80	F	67.4	A	3		HA	INSTRU	STATOR WATER COOLING FLOW SWITCH FAILURE.

SUMMARY

SEVERAL LOAD REDUCTIONS OCCURRED THROUGHOUT THE REPORT PERIOD FOR CONDENSER PUMP SUCTION STRAINER CLEANING, BACKFLUSHING OF THE NOS. 11 AND 12 HEATER DRAIN PUMPS, AND TO CLEAN THE NO. 13A CIRCULATING WATER PUMP WATERBOX. REACTOR TRIPS OCCURRED ON 02/14/80 WHEN THE FEEDWATER REGULATING VALVE FAILED SHUT, AND ON 02/26/80 FROM LOSS OF STATOR WATER COOLING TO THE GENERATOR.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

SAN ONOFRE 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	California/San Diego 5 miles S of San Clemente
2.	Electric Reliability Council	WSCC
3.	Type of Reactor	PWR
4.	Power Level (MWT)	1347
5.	Design Electrical Rating (MWE Net)	436
6.	Date of Initial Criticality	June 14, 1967
7.	Date of Commercial Operation	January 1, 1968
8.	Condenser Cooling Method	Circulating sea water
9.	Condenser Cooling Water Source	Pacific Ocean

Utility & Contractor Information

10.	Licensee	Southern California Edison Co.
11.	Corporate Address	2244 Walnut Grove Avenue Rosemead, California 91770
12.	Corporate Contact	R. E. Woodbury V.P. & General Counsel
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	V
17.	IE Branch Chief	J. L. Crews
18.	IE Senior Inspector	B. H. Faulkenberry
19.	IE Resident Inspector	R. Pate (714) 492-2641
20.	Licensing Project Manager	A. Burger
21.	Docket Number	50-206
22.	License Number/Date of Issue	DPR-13, 3/27/67
23.	Public Document Room	Mission Viejo Branch Library 24851 Chrisanta Drive Mission Viejo, California 92676

INSPECTION STATUS

ENFORCEMENT STATUS

FAILURE OF SECURITY EQUIPMENT TO ANNUNCIATE LOSS OF POWER.
(8001 DEFICIENCY)

COMMUNICATION ROOM UNLOCKED AND UNATTENDED.
(8001 INFRACTION)

INADEQUATE STAND-BY POWER FOR SECURITY EQUIPMENT.
(8001 INFRACTION)

VITAL AREA ALARMS NOT PERFORMANCE TESTED EVERY SEVEN DAYS, AND SEVERAL TAMPER ALARMS NOT TESTED EVERY 90 DAYS.
(8001 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST (CITE RO REPORT NUMBER, IF APP.)

SYSTEMS AND COMPONENT PROBLEMS:

None.

FACILITY ITEMS (PLANS AND PROCEDURES):

None.

MANAGERIAL ITEMS

None.

PLANT STATUS:

Plant operating at 100% power.

LAST IE SITE INSPECTION DATE: 01/28-02/07/80

INSPECTION REPORT NO: 50-206/80-03

SUMMARY

INSPECTION SUMMARY

- Inspection on January 7-11, 1980 (Report No. 50-206/80-01)
  Areas Inspected: Included review of: Security organization; protected area physical barriers; vital area physical barriers; lighting; access control (identification, authorization and badging); access control (search); access control (escorting); access control (vital areas); alarm stations; detection aids; assessment aids; communications; testing and maintenance; compensatory measures; power supply; response; locks; keys and cobminations; records and reports; followup on previous items of noncompliance and followup on IE Bulletin No. 79-16. The inspection involved 108 inspector hours by three NRC inspectors.

  Results: The licensee was found to be in compliance with NRC requirements within the 20 areas examined during this inspection except for the following four items: (1) Testing and Maintenance - Certain vital area door/gate alarms and certain tamper alarms were not performance tested as required - Infraction; (2) Power Supply - Some security equipment failed to operate when transferred from 118 volt primary power to backup power - Infraction; (3) Communications - A communication room was found unlocked and unattended - Infraction; Power Supply - The loss of 118 volt primary power failed to annunciate as required - Deficiency.

- Inspection on January 7-25, 1980 (Report No. 50-206/80-02)
  Areas Inspected: Routine resident inspection of plant operations. The inspection involved 55 inspector hours by one NRC Resident Inspector.

  Results: No items of noncompliance or deviations were identified.

- Inspection on January 28 - February 7, 1980 (Report No. 50-206/80-03)
  Report being prepared; to be reported next month.

REPORTS RECEIVED FROM LICENSEE

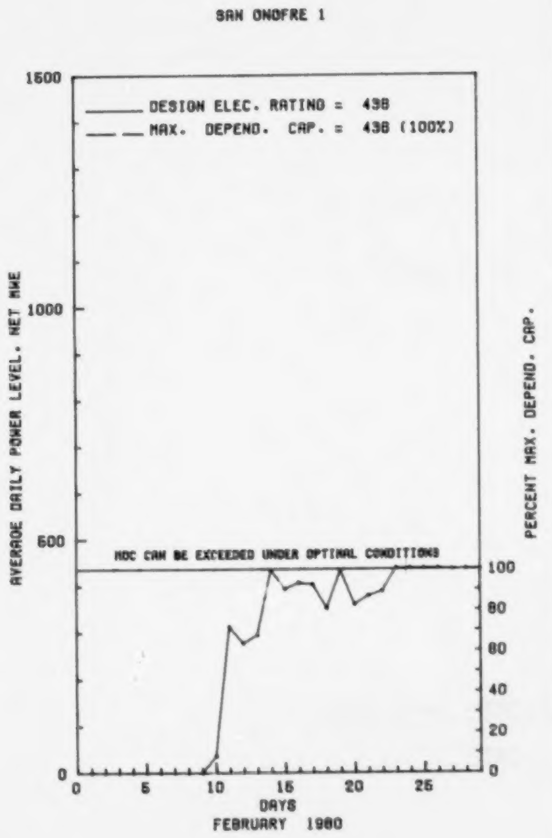
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-24-L	12-03-79	01-31-80	30 Day LER	Modifications to plant equipment without approved design change.
80-03-L	01-09-80	02-04-80	30 Day LER	Service water containment isolation valve failed to close upon testing.
80-04-L	01-19-80	02-13-80	30 Day LER	Feedwater flow indication increased 15%; Mismatch channel inoperable.
80-05-L	02-18-80	02-28-80	14 Day LER	Condenser delta T exceeded environmental TS 2.2.1 limit.

REVIEWED BY OMPA LOEB: T.C. Cintula **DATE:** 4-10-80

REVIEWED BY NRR DOR: L. Burger **DATE:** 4-14-80

SAN ONOFRE 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-206
2. Utility Contact: P. Wetmon (714) 492-7700
3. Licensed Thermal Power (MWt): 1347
4. Nameplate Rating (Gross MWe): 500 x 0.9 = 450
5. Design Electrical Rating (Net MWe): 436
6. Maximum Dependable Capacity (Gross MWe): 456
7. Maximum Dependable Capacity (Net MWe): 436
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	695.0	1,440.0	111,416.0
12. Hours Reactor Was Critical	469.3	1,037.4	83,669.6
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	457.2	1,022.1	80,174.5
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	559,989	1,310,865	102,842,339
17. Gross Electrical Energy (MWH)	189,000	436,200	35,107,634
18. Net Electrical Energy (MWH)	180,014	414,205	33,263,571
19. Unit Service Factor	65.7	71.0	75.4
20. Unit Availability Factor	65.7	71.0	75.4
21. Unit Capacity Factor (MDC Net)	59.3	66.3	72.6
22. Unit Capacity Factor (DER Net)	59.3	66.3	72.6
23. Unit Forced Outage Rate	1.7	4.3	9.2

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
REFUELING APRIL 11, 1980 - 7 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
44	01/26/80	S	230.8	D	4		ZZ	ZZZZZZ	TMI MODIFICATIONS
45	02/11/80	F	41.1	A	5		HA	TURBIN	LOW GOVERNOR OIL PRESSURE
46	02/12/80	F	8.0	A	1		HA	TURBIN	REPAIRED GOVERNOR CONTROL OIL SYSTEM.
47	02/15/80	S	7.5	B	5		HC	HTEXCH	CLEAN CONDENSER WATER BOXES.
48	02/17/80	F	5.7	B	5		HC	HTEXCH	REPAIRED SALT WATER LEAKS BY PLUGGING CONDENSER TUBES.
49	02/18/80	S	12.0	B	5		HC	HTEXCH	CLEAN CONDENSER WATER BOXES.
50	02/19/80	S	6.8	B	5		HC	HTEXCH	CLEAN CONDENSER WATER BOXES.
51	02/20/80	S	5.5	B	5		HC	HTEXCH	CLEAN CONDENSER WATER BOXES.
52	02/20/80	S	7.7	B	5		HC	HTEXCH	CLEAN CONDENSER WATER BOXES.
53	02/21/80	S	8.5	B	5		HC	HTEXCH	CLEAN CONDENSER WATER BOXES.

SUMMARY

THE UNIT RETURNED TO POWER ON 02/10/80 FOLLOWING COMPLETION OF TMI-RELATED MODIFICATIONS AND A GENERAL PLANT MAINTENANCE OUTAGE. IT WAS TAKEN OFF-LINE AGAIN ON 02/12/80 TO REPAIR THE GOVERNOR OIL PRESSURE. SEVERAL LOAD REDUCTIONS OCCURRED FOR CONDENSER WATERBOX CLEANING, AND TO REPAIR A SALTWATER LEAK IN THE CONDENSER.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

ST. LUCIE 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Florida/St. Lucie 12 miles SE of Ft. Pierce
2.	Electric Reliability Council	Southern
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2560
5.	Design Electrical Rating (MWE Net)	802
6.	Date of Initial Criticality	April 22, 1976
7.	Date of Commercial Operation	December 21, 1976
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Atlantic Ocean

Utility & Contractor Information

10.	Licensee	Florida Power & Light Co.
11.	Corporate Address	9250 W. Flagler Street P. O. Box 529100 Miami, Florida 33152
12.	Corporate Contact	R. E. Uhrig, V.P.
13.	Architect/Engineer	Ebasco
14.	Nuclear Steam Supply System	Combustion Engineering
15.	Constructor	Ebasco

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. D. MARTIN
18.	IE Senior Inspector	E.H. Verdery
19.	IE Principal Inspector	S. A. ELROD
20.	Licensing Project Manager	P. B. Erickson
21.	Docket Number	50-335
22.	License Number/Date of Issue	DPR-67, March 1, 1976
23.	Public Document Room	Indian River Junior College Library 3209 Virginia Avenue Ft. Pierce, Florida 33450

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

◆ REFUELING OUTAGE SCHEDULED TO BEGIN MARCH 15, 1980.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

UNIT OPERATING AT 100% POWER.

◆ LAST IE SITE INSPECTION DATE: FEBRUARY 26-29, 1980

◆ INSPECTION REPORT NO: 50-335/80-05

SUMMARY

INSPECTION SUMMARY

◆ INSPECTION JANUARY 22-24 (80-03): THIS SPECIAL, ANNOUNCED INSPECTION INVOLVED 38 INSPECTOR-HOURS ON SITE IN THE AREAS OF SMALL BREAK LOSS OF COOLANT PROCEDURES AND TRAINING, AND FOLLOWUP ON PREVIOUS INSPECTOR IDENTIFIED ITEMS. WITHIN THE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

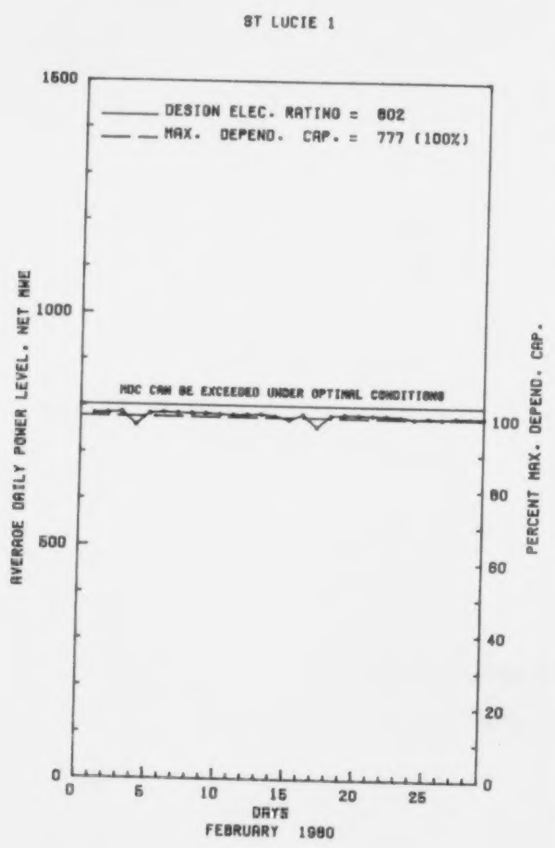
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
80-01/ 03L-0	01/02/80	02/04/80	30 DAY LER	1A DIESEL FUEL TRANSFER PUMP FAILED TO START.
80-02/ 03L-0	01/04/80	02/04/80	30 DAY LER	CEA 22 DROPPED DUE TO FAILED POWER SUPPLY.
80-03/ 03L-0	01/06/80	02/05/80	30 DAY LER	CEA 30 REED SWITCH POSITION INDICATION CHANNEL FAILED.
80-04/ 01T-0	01/14/80	01/25/80	14 DAY LER	FAILURE TO MEET LCO FOR OPERABLE AC ELECTRICAL POWER SOURCES.

REVIEWED BY OMPA LOEB: Mk Beebe **DATE:** 04/10/80

REVIEWED BY NRR DOR: P. B. [illegible] **DATE:** [illegible]

ST. LUCIE 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-335
2. Utility Contact: V. T. Chilson (305) 552-3824
3. Licensed Thermal Power (MWt): 2560
4. Nameplate Rating (Gross MWe): 1000 x 0.85 = 850
5. Design Electrical Rating (Net MWe): 802
6. Maximum Dependable Capacity (Gross MWe): 822
7. Maximum Dependable Capacity (Net MWe): 777
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	696.0	1,440.0	27,984.0
12. Hours Reactor Was Critical	696.0	1,426.4	22,936.3
13. Reactor Reserve Shutdown Hours	.0	.0	129.4
14. Hours Generator On-Line	696.0	1,423.9	22,254.0
15. Unit Reserve Shutdown Hours	.0	0	32.0
16. Gross Thermal Energy (MWH)	1,775,685	3,594,611	53,854,177
17. Gross Electrical Energy (MWH)	573,870	1,163,040	17,457,540
18. Net Electrical Energy (MWH)	543,137	1,099,778	16,427,674
19. Unit Service ractor	100.0	98.9	79.5
20. Unit Availability Factor	100.0	98.9	79.6
21. Unit Capacity Factor (MDC Net)	108.4	98.3	75.6
22. Unit Capacity Factor (DER Net)	97.3	95.2	73.2
23. Unit Forced Outage Rate	.0	1.1	5.1

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUELING, MAINT. & INSPECT. - MARCH 15 - MAY 17, 1980.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	NONE								

SUMMARY

OPERATION WAS AT NEAR FULL POWER THROUGHOUT THE REPORT PERIOD, WITH NO SHUTDOWNS OR REDUCTIONS REPORTED.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

SURRY 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1. Location (state/county) (nearest population center) Virginia/Surry 17 miles NW of Newport News
2. Electric Reliability Council Southeastern
3. Type of Reactor PWR
4. Power Level (MWT) 2441
5. Design Electrical Rating (MWE Net) 822
6. Date of Initial Criticality July 1, 1972
7. Date of Commercial Operation December 22, 1972
8. Condenser Cooling Method Once Through
9. Condenser Cooling Water Source James River

Utility & Contractor Information

10. Licensee Virginia Electric and Power Company
11. Corporate Address P. O. Box 26666 Richmond, Virginia 23261
12. Corporate Contact J. H. Ferguson Exec. V.P. - Power
13. Architect/Engineer Stone & Webster
14. Nuclear Steam Supply System Westinghouse
15. Constructor Stone & Webster

Regulatory Information

16. IE Region responsible II
17. IE Branch Chief R.C. Lewis
18. IE Senior Inspector P. J. Kellogg
19. IE Resident Inspector D. Burke (804) 357-2101
20. Licensing Project Manager D. Neighbors
21. Docket Number 50-280
22. License Number/Date of Issue DPR-32, 5/25/72
23. Public Document Room Swem Library College of William and Mary Williamsburg, Virginia 23185

INSPECTION STATUS

ENFORCEMENT STATUS

AS REQUIRED BY TECHNICAL SPECIFICATION 6.4.A.7 AND 6.4.D, THE DETAILED WRITTEN PROCEDURES PROVIDED FOR PREVENTIVE OR CORRECTIVE MAINTENANCE OPERATIONS WHICH WOULD HAVE AN EFFECT ON THE SAFETY OF THE REACTOR MUST BE FOLLOWED. ON NOVEMBER 20, 1979, THE INSPECTOR OBSERVED THAT THE HEAT-TRACED UNIT 1 EMERGENCY BORATE LINE (2" CH-42-152) WAS NOT PROPERLY INSULATED, INDICATING THAT CORRECTIVE MAINTENANCE PROCEDURES EMP-C-HT-20 OR MMP-C-G-017, WHICH REQUIRES REINSTALLATION OF INSULATION REMOVED, WERE NOT FOLLOWED. THE MISSING INSULATION MAY HAVE LED TO OR AGGRAVATED THE BORATE LINE PLUGGING.
(7967 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

RE-ANALYSIS AND REVIEW OF SEISMIC DESIGN DEFICIENCIES ARE CONTINUING.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

+ UNIT IS CURRENTLY SHUTDOWN FOR MAIN TURBINE INSPECTION AND CORRECTION OF SEISMIC DESIGN ERRORS.

+ LAST IE SITE INSPECTION DATE: FEBRUARY 29, 1980

+ INSPECTION REPORT NO: 50-280/80-05

SUMMARY

INSPECTION SUMMARY

+ INSPECTION JANUARY 29 - FEBRUARY 1 (80-02): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 32 INSPECTOR-HOURS ON SITE IN THE AREAS OF LICENSEE ACTION ON PREVIOUS INSPECTION FINDINGS, CONSTRUCTION TESTING, REVIEW OF QUALITY RECORDS, PRIMARY AND SECONDARY HYDROSTATIC TEST AND CONSTRUCTION STATUS. OF THE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

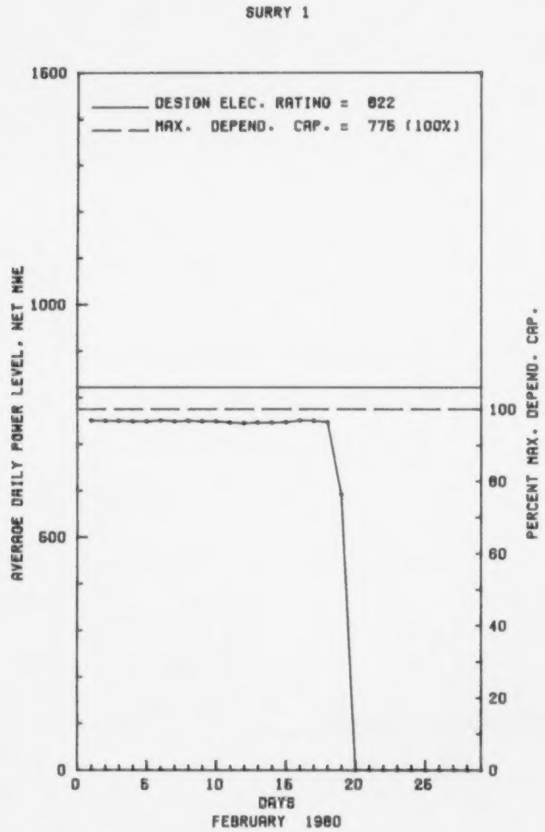
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
80-001/ 03L-0	01/01/80	01/22/80	30 DAY LER	MAIN FEED SYSTEM SNUBBER 1-WFPD-HSS-14 FOUND INOPERABLE.
80-002/ 03L-0	01/06/80	01/25/80	30 DAY LER	EFFLUENT FLOW RECORDER FAILED DURING A ROUTINE LIQUID RELEASE.
80-003/ 03L-0	01/07/80	01/25/80	30 DAY LER	FEEDWATER BYPASS VALVE HCV-FW-155B DID NOT FUNCTION PROPERLY DURING TESTING.

REVIEWED BY OMPA LOEB: **DATE:** 4/10/80

REVIEWED BY NRR DOR: **DATE:** 4/11/80

SURRY 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-280
2. Utility Contact: O. J. Costello (804) 357-3184
3. Licensed Thermal Power (MWt): 2441
4. Nameplate Rating (Gross MWe): 942 x 0.9 = 848
5. Design Electrical Rating (Net MWe): 822
6. Maximum Dependable Capacity (Gross MWe): 811
7. Maximum Dependable Capacity (Net MWe): 775
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	696.0	1,440.0	63,024.0
12. Hours Reactor Was Critical	454.8	1,016.2	39,767.8
13. Reactor Reserve Shutdown Hours	.0	182.6	3,731.5
14. Hours Generator On-Line	454.3	1,010.4	38,915.4
15. Unit Reserve Shutdown Hours	.0	183.6	3,736.2
16. Gross Thermal Energy (MWH)	1,100,867	2,438,340	90,153,212
17. Gross Electrical Energy (MWH)	355,470	785,655	29,466,153
18. Net Electrical Energy (MWH)	337,791	746,810	27,973,709
19. Unit Service Factor	65.3	70.2	61.7
20. Unit Availability Factor	65.3	82.9	67.7
21. Unit Capacity Factor (MDC Net)	62.6	66.9	57.3
22. Unit Capacity Factor (DER Net)	59.0	63.1	54.0
23. Unit Forced Outage Rate	34.7	29.8	24.9

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUELING - JUNE 1980 - 6 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: 05-10-80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
8-4	02/19/80	F	241.7	D	1	80-018		PIPEXX	PIPE STRESS REANALYSIS ON REACTOR CAVITY PURIFICATION AND SEAL WATER RETURN LINES REVEALED A POTENTIAL OVERSTRESSED CONDITION ON THE PIPING PENETRATIONS IN THE EVENT OF A DESIGN BASIS EARTHQUAKE. A TURBINE INSPECTION WAS COMMENCED IMMEDIATELY AFTER SHUTDOWN DUE TO ABOVE NORMAL VIBRATION EXPERIENCED IMMEDIATELY PRIOR TO UNIT SHUTDOWN AND TO CONDUCT INSPECTIONS OF THE LOW PRESSURE ROTOR DISCS. IE BULLETIN 79-14.

SUMMARY

OPERATION WAS ROUTINE UNTIL 02/19/80, WHEN THE UNIT WAS TAKEN OFF-LINE FOR PIPE STRESS ANALYSIS AND TURBINE INSPECTION.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

SURRY 2

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Virginia/Surry 17 miles NW of Newport News
2.	Electric Reliability Council	Southeastern
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2441
5.	Design Electrical Rating (MWE Net)	822
6.	Date of Initial Criticality	March 7, 1973
7.	Date of Commercial Operation	May 1, 1973
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	James River

Utility & Contractor Information

10.	Licensee	Virginia Electric and Power Company
11.	Corporate Address	P. O. Box 26666 Richmond, Virginia 23261
12.	Corporate Contact	J. H. Ferguson Exec. V.P. - Power
13.	Architect/Engineer	Stone & Webster
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R.C. Lewis
18.	IE Senior Inspector	P. J. Kellog
19.	IE Resident Inspector	D. Burke (804) 357-2101
20.	Licensing Project Manager	D. Neighbors
21.	Docket Number	50-281
22.	License Number/Date of Issue	DPR-37, 1/29/73
23.	Public Document Room	Swem Library College of William and Mary Williamsburg, Virginia 23185

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

SYSTEM MODIFICATIONS FOR SEISMIC SUPPORT REANALYSIS AND NUREG 0578 ARE CONTINUING.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

UNIT IS CURRENTLY SHUT DOWN AND DEFUELED. THE STEAM GENERATOR REPLACEMENT PROJECT COMPLETED.

+ LAST IE SITE INSPECTION DATE: FEBRUARY 15, 1980

+ INSPECTION REPORT NO: 50-281/80-05

SUMMARY

INSPECTION SUMMARY

+ INSPECTION JANUARY 29 - FEBRUARY 1 (80-04): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 32 INSPECTOR-HOURS ON SITE IN THE AREAS OF LICENSEE ACTION ON PREVIOUS INSPECTION FINDINGS, CONSTRUCTION TESTING, REVIEW OF QUALITY RECORDS, PRIMARY AND SECONDARY HYDROSTATIC TEST AND CONSTRUCTION STATUS. OF THE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

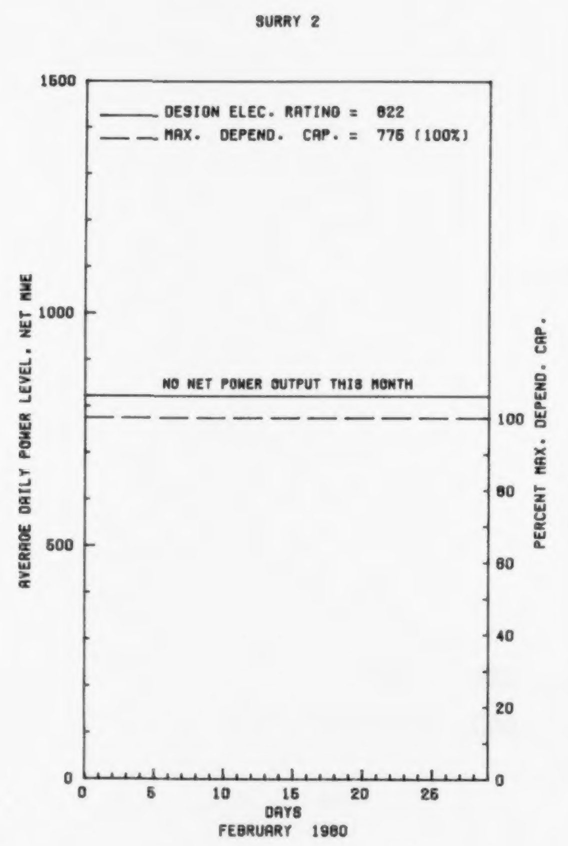
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
none				

REVIEWED BY OMPA LOEB: **DATE:** 4/10/80

REVIEWED BY NRR DOR: **DATE:** 4/11/80

SURRY 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-281
2. Utility Contact: O. J. Costello (804) 357-3184
3. Licensed Thermal Power (MWt): 2441
4. Nameplate Rating (Gross MWe): 942 x 0.9 = 848
5. Design Electrical Rating (Net MWe): 822
6. Maximum Dependable Capacity (Gross MWe): 811
7. Maximum Dependable Capacity (Net MWe): 775
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	696.0	1,440.0	59,909.0
12. Hours Reactor Was Critical	.0	.0	34,499.9
13. Reactor Reserve Shutdown Hours	.0	.0	0
14. Hours Generator On-Line	.0	.0	33,996.2
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	0	79,194,083
17. Gross Electrical Energy (MWH)	0	0	25,868,844
18. Net Electrical Energy (MWH)	0	0	24,535,605
19. Unit Service Factor	.0	.0	56.8
20. Unit Availability Factor	.0	.0	56.8
21. Unit Capacity Factor (MDC Net)	.0	.0	52.9
22. Unit Capacity Factor (DER Net)	.0	.0	49.8
23. Unit Forced Outage Rate	.0	.0	21.0

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 05/05/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-2	02/04/79	S	696.0	C	4			HTEXCH	CONTINUATION OF SHUTDOWN FOR REFUELING AND STEAM GENERATOR REPLACEMENT WHICH BEGAN ON 02/04/79.

SUMMARY

THE UNIT REMAINED SHUTDOWN FOLLOWING STEAM GENERATOR REPLACEMENT.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

THREE MILE ISLAND 1

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Pennsylvania/Dauphin 10 miles SE of Harrisburg
2.	Electric Reliability Council	MAAC
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2535
5.	Design Electrical Rating (MWE Net)	819
6.	Date of Initial Criticality	June 5, 1974
7.	Date of Commercial Operation	September 2, 1974
8.	Condenser Cooling Method	Closed
9.	Condenser Cooling Water Source	Susquehanna River

Utility & Contractor Information

10.	Licensee	Metropolitan Edison Company
11.	Corporate Address	P. O. Box 542 Reading, Pennsylvania
12.	Corporate Contact	R. C. Arnold, Sr. Vice-President 260 Cherry Hill Rd. Parsippany, NJ 07054
13.	Architect/Engineer	Gilbert Associates, Inc.
14.	Nuclear Steam Supply System	B&W
15.	Constructor	UE&C

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Senior Inspector	R. R. Keimig
19.	NRC-TMI Support Staff	D. Haverkamp (717) 782-3950
20.	Licensing Project Manager	D. C. DiIanni
21.	Docket Number	50-289
22.	License Number/Date of Issue	DPR-50, 4/19/74
23.	Public Document Room	Government Publications Section State Library of Pennsylvania Education Building Commonwealth and Walnut Street Harrisburg, Pennsylvania 17126

INSPECTION STATUS

ENFORCEMENT STATUS

CHANGES WERE MADE TO THE OPERATIONAL QUALITY ASSURANCE PLAN WHICH CHANGED THE QA PROGRAM FOR OPERATIONS AS DESCRIBED IN THE FSAR SECTION 17.2. HOWEVER, THESE CHANGES WERE NOT SUBMITTED TO THE NRC IN THE SUBSEQUENT ANNUAL REPORT. THIS IS CONTRARY TO 10 CFR 50.59(B).
(7916 INFRACTION)

PURCHASE ORDERS FOR RESTART MODIFICATIONS TO UNIT 1 WERE ISSUED IN ACCORDANCE WITH A FIELD PURCHASING PROCEDURE WHICH WAS NOT APPROVED. THIS IS CONTRARY TO 10 CFR 50, APPENDIX B, CRITERION V, AND THE QA PLAN FOR RESTART MODIFICATIONS TO UNIT 1.
(7916 DEFICIENCY)

RESTART MODIFICATION RM-1 WAS IMPLEMENTED WITH DRAWINGS WHICH WERE NOT APPROVED FOR CONSTRUCTION. THIS IS CONTRARY TO 10 CFR 50, APPENDIX B, CRITERION VI, AND MET-ED. PROCEDURE GP-1003.
(7916 INFRACTION)

THE DISTRIBUTION OF DRAWINGS AND THE ELIMINATION OF OBSOLETE DRAWINGS WAS NOT BEING CONTROLLED. THIS IS CONTRARY TO 10 CFR 50, APPENDIX B, CRITERION VI, AND THE FSAR SECTION 17.2.11.
(7916 INFRACTION)

SITE QC WAS NOT PERFORMING PERIODIC SURVEILLANCE OF WAREHOUSE STORAGE AREAS AS REQUIRED BY THE TMI OQA PLAN, AND MET-ED PROCEDURES AP1018 AND GP4014. GPUSC QA WAS NOT PERFORMING REGULARLY SCHEDULED INSPECTIONS OF UNIT 1 RESTART MODIFICATION MATERIAL IN STORAGE AS REQUIRED BY THE QA PLAN FOR RESTART MODIFICATIONS TO UNIT 1. THIS IS CONTRARY TO 10 CFR 50, APPENDIX B, CRITERION XIII, ANSI N45.2.2-1972, AND THE PROCEDURES STATED ABOVE.
(7916 DEFICIENCY)

CORRECTIVE ACTION WAS NOT PERFORMED IN A TIMELY MANNER SUBSEQUENT TO THE CONDUCT OF INTERNAL AUDITS. THIS IS CONTRARY TO 10 CFR 50, APPENDIX B, CRITERION XVI, AND THE OQA PLAN DESCRIBED IN UNIT 2 FSAR, SECTION 17.2.23.
(7916 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

None

FACILITY ITEMS (PLANS AND PROCEDURES):

None

MANAGERIAL ITEMS:

Mr. R. M. Toole was appointed to the new position of TMI-1 Manager effective February 1, 1980.

PLANT STATUS:

The plant remains shutdown by NRC order, pending completion of modifications and other actions related to the TMI-2 accident.

LAST IE SITE INSPECTION DATE: 2/20/80

INSPECTION REPORT NO: 50-289/80-02

SUMMARY

INSPECTION SUMMARY

No inspection reports issued.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT

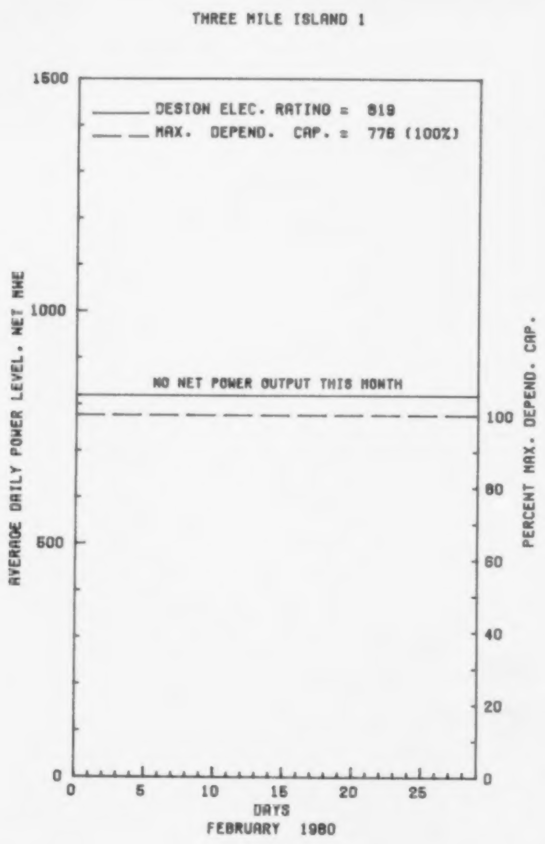
NONE

REVIEWED BY OMPA LOEB: E Bobe DATE: 4/10/80

REVIEWED BY NRR DOR: D C D. DATE: 4/11/80

THREE MILE ISLAND 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-289
2. Utility Contact: D. G. Mitchell (215) 921-6579
3. Licensed Thermal Power (MWt): 2535
4. Nameplate Rating (Gross MWe): 968 x 0.9 = 871
5. Design Electrical Rating (Net MWe): 819
6. Maximum Dependable Capacity (Gross MWe): 840
7. Maximum Dependable Capacity (Net MWe): 776
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	696.0	1,440.0	48,169.0
12. Hours Reactor Was Critical	.0	.0	31,731.8
13. Reactor Reserve Shutdown Hours	.0	.0	838.5
14. Hours Generator On-Line	.0	.0	31,180.9
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	0	76,531,071
17. Gross Electrical Energy (MWH)	0	0	25,484,330
18. Net Electrical Energy (MWH)	0	0	23,840,053
19. Unit Service Factor	.0	.0	64.7
20. Unit Availability Factor	.0	.0	64.7
21. Unit Capacity Factor (MDC Net)	.0	.0	63.8
22. Unit Capacity Factor (DER Net)	.0	.0	60.4
23. Unit Forced Outage Rate	100.0	100.0	23.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: UNKNOWN

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
1	02/17/79	F	696.0	D	4				REGULATORY RESTRAINT ORDER

SUMMARY

THE UNIT REMAINS SHUTDOWN DUE TO REGULATORY CONSIDERATION OF THE TMI-2 ACCIDENT.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

THREE MILE ISLAND 2

NRC POWER RESTRICTIONS: SEE "LICENSE STATUS"

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Pennsylvania/Dauphin 10 miles SE of Harrisburg
2.	Electric Reliability Council	MAAC
3.	Type of Reactor	PWR
4.	Licenses Power Level (MWT)	2772
5.	Design Electrical Rating (MWE Net)	906
6.	Date of Initial Criticality	3/28/78
7.	Date of Commercial Operation	12/30/78
8.	Condenser Cooling Method	CLOSED
9.	Condenser Cooling Water Source	Susquehanna River

Utility & Contractor Information

10.	Licensee	Metropolitan Edison Company
11.	Corporate Address	P. O. Box 542 Reading, PA 19603
12.	Corporate Contact	R. C. Arnold, Sr. Vice-President 260 Cherry Hill Rd. Parsippany, NJ 07054
13.	Architect/Engineer	Burns & Roe
14.	Nuclear Steam Supply System	B&W
15.	Constructor	UE&C

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Senior Inspector	R. R. Keimig
19.	NRC-TMI Support Staff	R. Conte (717) 782-3950
20.	Licensing Project Manager	R. VOLLMER
21.	Docket Number	50-320
22.	License Number/Date of Issue	DPR-73/2; 2/8/78
23.	Public Document Room	Government Publications Section State Library of Pennsylvania Education Building Commonwealth & Walnut Streets Harrisburg, Pennsylvania 17126

INSPECTION STATUS

ENFORCEMENT STATUS

CHANGES WERE MADE TO THE OPERATIONAL QUALITY ASSURANCE PLAN WHICH CHANGED THE QA PROGRAM FOR OPERATIONS AS DESCRIBED IN THE FSAR, SECTION 17.2. HOWEVER, THESE CHANGES WERE NOT SUBMITTED TO THE NRC IN THE SUBSEQUENT ANNUAL REPORT. THIS IS CONTRARY TO 10 CFR 50.59(B).
(7920 INFRACTION)

MATERIAL PURCHASED FOR AND MODIFICATION PERFORMED ON A QA COGNIZANT SYSTEM WITHOUT QA/QC INVOLVEMENT. THIS IS CONTRARY TO 10 CFR 50, APPENDIX B, CRITERION II AND MET-ED PROCEDURE GP-1008.
(7920 INFRACTION)

PROCEDURES WERE NOT REVIEWED WITHIN THE REQUIRED TIME FRAME. THIS IS CONTRARY TO 10 CFR 50, APPENDIX B, CRITERION VI, THE TMI OQA PLAN, ANSI N18.7-1976, AND MET-ED PROCEDURE AP1001.
(7920 DEFICIENCY)

THE DISTRIBUTION OF DRAWINGS AND THE ELIMINATION OF OBSOLETE DRAWINGS WAS NOT BEING CONTROLLED. THIS IS CONTRARY TO 10 CFR 50, APPENDIX B, CRITERION VI, AND THE FSAR, SECTION 17.2.11.
(7920 INFRACTION)

GPUSC - SITE QA WAS NOT PERFORMING MONTHLY SURVEILLANCES OF STORAGE AREAS. THIS IS CONTRARY TO GPUSC PROCEDURE QCP-M-002, STORAGE CONTROL, AND 10 CFR 50, APPENDIX B, CRITERION XIII.
(7920 DEFICIENCY)

CORRECTIVE ACTION WAS NOT PERFORMED IN A TIMELY MANNER SUBSEQUENT TO THE CONDUCT OF INTERNAL AUDITS. THIS IS CONTRARY TO 10 CFR 50, APPENDIX B, CRITERION XVI, AND THE UNIT 2 FSAR, SECTION 17.2.23.
(7920 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

Recovery operations from the 3/28/79 incident.

FACILITY ITEMS (PLANS AND PROCEDURES):

None

MANAGERIAL ITEMS:

None

PLANT STATUS:

No significant information to report.

LAST IE SITE INSPECTION DATE: 1/31/80

INSPECTION REPORT NO: 50-320/80-01

SUMMARY

INSPECTION SUMMARY

No inspections completed.

LICENSE STATUS

By Order for Modification of License dated July 20, 1979, the licensee's authority to operate the facility was suspended and the licensee's authority was limited to maintenance of the facility in the present shutdown cooling mode (44 F.R. 45271).

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT

NONE

REVIEWED BY OMPA LOEB: E Bobe **DATE:** 4/10/80

REVIEWED BY NRR DOR: **DATE:** 4/[illegible]

THREE MILE ISLAND 2

AVERAGE DAILY POWER LEVEL (MWe)

OPERATING STATUS

1. Docket Number: 50-320
2. Utility Data Prepared By: R. A. Lengel (215) 921-6581
3. Licensed Thermal Power (MWt): 2772
4. Nameplate Rating (Gross MWe): 1068 X 0.9 = 961
5. Design Electrical Rating (Net MWe): 906
6. Maximum Dependable Capacity (Gross MWe): 0
7. Maximum Dependable Capacity (Net MWe): 0
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons
None
9. Power Level To Which Restricted, If Any (Net MWe): None*
10. Reasons For Restrictions, If Any:

This Month Yr.-to

11. Hours In Reporting Period
12. Number Of Hours Reactor Was Critical
13. Reactor Reserve Shutdown Hours
14. Hours Generator On-Line
15. Unit Reserve Sh
16. Gro
(MWH)
19
20. U Availability Factor
21. Unit Capacity Factor (Using MDC Net) 1/
22. Unit Capacity Factor (Using DER Net) 1/
23. Unit Forced Outage Rate
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
None
25. If Shut Down At End Of Report Period, Estimated Date of Startup: N/A

NO DECISION ON FUTURE OPERATION
SHUTDOWN INDEFINITELY

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence

SEE SUMMARY BELOW.

SUMMARY

THE UNIT HAS REMAINED SHUTDOWN SINCE THE 03/28/79 ACCIDENT, WHICH CAUSED EXTENSIVE CORE DAMAGE.

AUTHORITY TO OPERATE FACILITY SUSPENDED BY NRC ORDER DATED JULY 20, 1979, WHICH REQUIRES LICENSEE TO MAINTAIN FACILITY IN A SHUTDOWN CONDITION.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

TROJAN

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Oregon/Columbia 42 miles N of Portland
2.	Electric Reliability Council	WSCC
3.	Type of Reactor	PWR
4.	Licenses Power Level (MWT)	3411
5.	Design Electrical Rating (MWE Net)	1130
6.	Date of Initial Criticality	December 15, 1975
7.	Date of Commercial Operation	May 20, 1976
8.	Condenser Cooling Method	Natural draft cooling tower
9.	Condenser Cooling Water Source	Columbia River

Utility & Contractor Information

10.	Licensee	Portland General Electric Co.
11.	Corporate Address	121 S. W. Salmon St. Portland, Oregon 97204
12.	Corporate Contact	C. Goodwin, Jr., Asst. V.P.
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Multiple

Regulatory Information

16.	IE Region responsible	V
17.	IE Branch Chief	J. L. Crews
18.	IE Senior Inspector	D. M. Sternberg
19.	IE Resident Inspector	M. H. Malmros (503) 556-3751
20.	Licensing Project Manager	C. M. Trammell
21.	Docket Number	50-344
22.	License Number/Date of Issue	NPF-1, 11/21/75
23.	Public Document Room	Columbia County Courthouse Law Library Circuit Court Room St. Helens, Oregon 97501

INSPECTION STATUS

ENFORCEMENT STATUS

FACILITY OPERATING LICENSE NO. NPF-1 REQUIRES IN SECTION 2.D THAT "THE LICENSEE SHALL MAINTAIN IN EFFECT AND FULLY IMPLEMENT ALL PROVISIONS OF THE COMMISSION - APPROVED PHYSICAL SECURITY PLAN, INCLUDING AMENDMENTS AND CHANGES MADE PURSUANT TO THE AUTHORITY OF 10 CFR 50.54(P)." CONTRARY TO THIS, THE REQUIREMENTS OF THE PHYSICAL SECURITY PLAN (SECTION 1.6.2.7(1)) WERE NOT ADHERED TO IN THAT AN ONSITE VEHICLE WAS LEFT UNATTENDED IN AN UNSECURED CONDITION.
(8001 INFRACTION)

CONTRARY TO PARAGRAPH 5.A OF APPENDIX A TO 10 CFR 55 AND PARAGRAPH 7.1.7 OF TROJAN TRAINING PROCEDURE TP-2-2, WHICH IS THE LICENSEE'S NRC-APPROVED OPERATOR RETRAINING PROGRAM, THE LICENSEE DOCUMENTED ONLY ONE OF THE 10 OR MORE REACTIVITY CONTROL MANIPULATIONS PERFORMED BY ONE LICENSED SENIOR OPERATOR DURING HIS TWO-YEAR LICENSE TERM ENDING IN OCTOBER 1979.
(8002 DEFICIENCY)

CONTRARY TO THE LICENSEE'S COMMITMENT EXPRESSED IN PARAGRAPH 13.0 OF TROJAN NUCLEAR PLANT TRAINING PROCEDURE TP-2-2, WHICH IS THE LICENSEE'S NRC-APPROVED OPERATOR RETRAINING PROGRAM, THE TRAINING SUPERVISOR DID NOT PREPARE QUARTERLY SUMMARIES OF THE RETRAINING PROGRAM (INTENDED TO INCLUDE A GENERAL PROGRESS REPORT ON THE RETRAINING PROGRAM AND A LISTING OF ACTUAL OR POTENTIAL DIFFICULTIES WITH RESPECT TO MEETING ITS REQUIREMENTS).
(8002 DEVIATION)

CONTRARY TO THE LICENSEE'S COMMITMENT EXPRESSED IN PARAGRAPH 4.1.11 OF TROJAN NUCLEAR PLANT TRAINING PROCEDURE TP-2-2, WHICH IS THE LICENSEE'S NRC-APPROVED OPERATOR RETRAINING PROGRAM, THE OPERATIONS SUPERVISOR DID NOT REVIEW PARTICIPATION OF LICENSED REACTOR OPERATORS OR SENIOR REACTOR OPERATORS IN THE REACTIVITY CONTROL MANIPULATIONS PORTION OF THE RETRAINING PROGRAM.
(8002 DEVIATION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST (CITE RO REPORT NUMBER, IF APP.)

SYSTEMS AND COMPONENT PROBLEMS:

None.

FACILITY ITEMS (PLANS AND PROCEDURES):

+ 1. The phase two Public Hearing on control building modifications is planned for April 1980.

2. A two and one-half month refueling outage is scheduled to commence in mid-April 1980.

MANAGERIAL ITEMS:

None.

PLANT STATUS

+ The plant operated at power throughout the month except for a brief outage following a plant trip on February 5, 1980. The trip was caused by inadvertent grounding of an instrument bus. At the end of February, the plant was in coastdown at approximately 80% power.

LAST IE SITE INSPECTION DATE: 02/01-29/80+

INSPECTION REPORT NO: 50-344/80-04+

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
NONE				

SUMMARY

INSPECTION SUMMARY:

+ Inspection on January 2-31, 1980 (Report No. 50-344/80-01)
Areas Inspected: Routine inspections of plant operations, maintenance, physical security, calibration, requalification training, surveillance, and followup on a Licensee Event Report. The inspection involved 84 inspector hours by the NRC Resident Inspector.

Results: One item of noncompliance was identified (Infraction - Failure to comply with a physical security procedure for securing of onsite vehicles).

+ Inspection on February 15, 1980 (Report No. 50-344/80-03)
Report being prepared; to be reported next month.

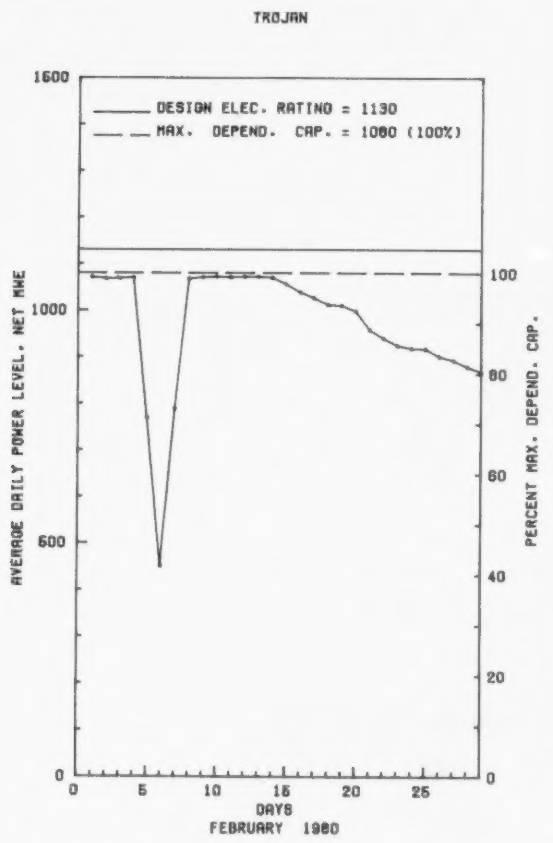
+ Inspection on February 1-29, 1980 (Report No. 50-344/80-04)
Report being prepared; to be reported next month.

REVIEWED BY OMPA LOEB: DATE: 4/10/80

REVIEWED BY NRR DOR: CM Trammell DATE: 4/11/80

TROJAN

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-344
2. Utility Contact: G. G. Bair (503) 556-3713 x234
3. Licensed Thermal Power (MWt): 3411
4. Nameplate Rating (Gross MWe): 1280 x 0.95 = 1216
5. Design Electrical Rating (Net MWe): 1130
6. Maximum Dependable Capacity (Gross MWe): 1122
7. Maximum Dependable Capacity (Net MWe): 1080
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	696.0	1,440.0	30,648.0
12. Hours Reactor Was Critical	694.5	1,420.3	17,718.3
13. Reactor Reserve Shutdown Hours	.0	.0	2,171.5
14. Hours Generator On-Line	693.6	1,415.4	17,043.2
15. Unit Reserve Shutdown Hours	.0	.0	1,508.7
16. Gross Thermal Energy (MWH)	2,159,205	4,556,572	53,452,182
17. Gross Electrical Energy (MWH)	706,175	1,484,595	17,470,010
18. Net Electrical Energy (MWH)	674,947	1,421,538	16,454,857
19. Unit Service Factor	99.7	98.3	55.6
20. Unit Availability Factor	99.7	98.3	60.5
21. Unit Capacity Factor (MDC Net)	89.8	91.4	49.7
22. Unit Capacity Factor (DER Net)	85.8	87.4	47.5
23. Unit Forced Outage Rate	.3	1.7	28.9

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUELING, APRIL 9, 1980; 83 DAYS.
25. If Shut Down At End of Report Period, Estimated Date of Sta-tup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-03	02/05/80	F	2.4	G	3			INSTRU	INADVERTENT GROUNDING OF PREFERRED INSTRUMENT AC BUS Y22-10 OCCURRED. THE GROUND RESULTED IN A RCP B BREAKER APPEARING TO OPEN, ALTHOUGH IN FACT IT WAS CLOSED, WHICH CAUSED REACTOR TO TRIP WHILE ABOVE P8.

SUMMARY

OPERATION WAS ROUTINE DURING THE REPORT PERIOD, WITH AN END -OF-CYCLE COASTDOWN BEGINNING 02/14/80. A REACTOR TRIP OCCURRED ON 02/05/80 WHEN INADVERTENT GROUNDING OF A PREFERRED INSTRUMENT AC BUS RESULTED IN A FALSE OPEN INDICATION IN A REACTOR COOLANT PUMP "B" BREAKER.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

TURKEY POINT 3

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Florida/Dade 25 miles S of Miami
2.	Electric Reliability Council	Southeastern
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2200
5.	Design Electrical Rating (MWE Net)	693
6.	Date of Initial Criticality	October 20, 1972
7.	Date of Commercial Operation	December 14, 1972
8.	Condenser Cooling Method	Recirculating Canal System
9.	Condenser Cooling Water Source	Biscayne Bay

Utility & Contractor Information

10.	Licensee	Florida Power and Light Company
11.	Corporate Address	9250 West Flagler Street P. O. Box 013100 Miami, Florida 33101
12.	Corporate Contact	R. E. Uhrig, V.P. Nuclear Affairs
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. Lewis
18.	IE Senior Inspector	E. H. Verdery
19.	IE Resident Inspector	R. J. Vogt-Lowell (305) 245-2910
20.	Licensing Project Manager	M. Grotenhuis
21.	Docket Number	50-250
22.	License Number/Date of Issue	DPR-31, 7/19/72
23.	Public Document Room	Environmental and Urban Affairs Library Florida International University Miami, Florida 33199

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO ETS SECTIONS 5.3.1, 5.3.2 AND 5.1, THE LICENSEE FAILED TO PROVIDE REQUIRED APPROVAL OF WRITTEN PROCEDURES FOR MONITORING, SURVEILLANCE, AND SPECIAL STUDY PROGRAMS DEFINED IN SECTION 4.0 OF THE ETS.
(7939 DEFICIENCY)

CONTRARY TO THE REQUIREMENTS OF 10 CFR 20.201(B)(1), ON DECEMBER 11, 1979 A CONTRACT WORKER MARKING THE TUBE SHEET OF STEAM GENERATOR "B" IN UNIT 3 WAS EXPOSED TO 3.55 REMS WHOLE BODY DOSE.
(7940 INFRACTION)

CONTRARY TO THE REQUIREMENTS OF TECH SPEC 6.8.1, PRIOR TO DECEMBER 11, 1979, NO PROCEDURE HAD BEEN ESTABLISHED FOR THE MARKING OF DEFECTIVE STEAM GENERATOR TUBES FOR SUBSEQUENT TUBE PLUGGING.
(7940 INFRACTION)

CONTRARY TO THE REQUIREMENTS OF TECHNICAL SPECIFICATION 6.8.1, THE REQUIREMENT OF RWP 79-681 WHERE NOT MET IN THAT 10 SEPARATE CHANNEL HEAD ENTRIES WERE MADE WITHOUT H.P. COVERAGE AND NO CHANNEL HEAD AIR SAMPLE WAS TAKEN DURING THE JOB.
(7940 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

NORMAL OPERATION AT 100% POWER.

LAST IE SITE INSPECTION DATE: FEBRUARY 28-29, 1980

INSPECTION REPORT NO: 50-250/80-11

SUMMARY

INSPECTION SUMMARY

INSPECTION FEBRUARY 5-7 (80-05): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 10 INSPECTOR-HOURS ON SITE IN THE AREAS OF PIPE SUPPORT BASEPLATE DESIGNS USING CONCRETE EXPANSION ANCHOR BOLTS (IEB 79-02); SEISMIC ANALYSIS FOR AS-BUILT SAFETY-RELATED PIPING SYSTEMS (IEB-79-14). OF THE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION DECEMBER 17-20 (79-40): THIS SPECIAL, UNANNOUNCED INSPECTION INVOLVED 13 1/2 INSPECTOR-HOURS ON SITE IN THE AREA OF REVIEW OF EVENTS SURROUNDING THE OVEREXPOSURE OF A CONTRACT WORKER. THREE APPARENT ITEMS OF NONCOMPLIANCE WERE FOUND IN THE ONE AREA INSPECTED (INFRACTION - EXCEEDED QUARTERLY WHOLE BODY RADIATION EXPOSURE LIMIT; INFRACTION - FAILURE TO HAVE A PROCEDURE; AND INFRACTION - FAILURE TO FOLLOW PROCEDURES).

INSPECTION JANUARY 8-11 (80-01): THIS ROUTINE UNANNOUNCED INSPECTION INVOLVED 17 INSPECTOR-HOURS ON SITE IN THE AREAS OF INSERVICE-INSPECTION DATA REVIEW AND EVALUATION; WELDING AND NONDESTRUCTIVE TESTING; IE BULLETIN 79-13, RADIOGRAPHY (UNIT 3); LP INSPECTION OF PRESSURIZER NOZZLE/SAFE END WELDS; EDDY CURRENT TESTING AND PLUGGING OF UNIT 3 STEAM GENERATORS. OF THE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-040/ 01T-0	12/12/79	12/28/79	14 DAY LER	INCORRECT VALVE WEIGHTS USED IN SEISMIC ANALYSIS OF FEEDWATER LINE FLOW REGULATOR BYPASS.
80-001/ 03L-0	01/03/80	02/04/80	30 DAY LER	FIRE PUMP LEFT INOPERABLE AFTER MAINTENANCE.

INSPECTION JANUARY 18-21 (80-02): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 7 INSPECTOR-HOURS ON SITE IN THE AREAS OF ENGINEERED SAFEGUARDS AND EMERGENCY POWER SYSTEMS INTEGRATED TESTING AND PERSONNEL AIRLOCK TESTING. OF THE TWO AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

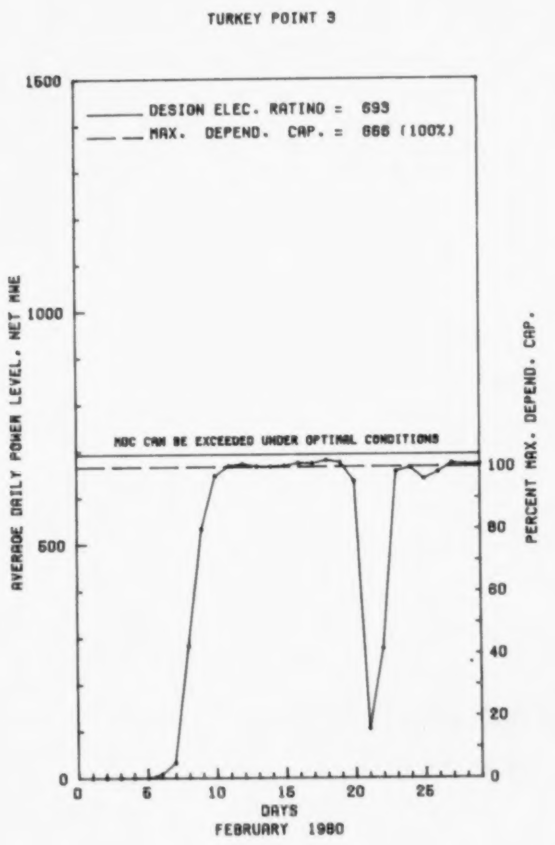
INSPECTION JANUARY 7-11 (80-03): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 13 1/2 INSPECTOR-HOURS ON SITE IN THE AREA OF UPGRADING OF THE TURKEY POINT FIRE PROTECTION PROGRAM. OF THE AREA INSPECTED, NO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED, BUT ONE APPARENT DEVIATION WAS FOUND (DEVIATION - FAILURE TO IMPLEMENT A FIRE PROTECTION QUALITY ASSURANCE PROGRAM).

REVIEWED BY OMPA LOEB: T.C. CINTULA **DATE:** 4·10·80

REVIEWED BY NRR DOR: **DATE:** 4·11·8

TURKEY POINT 3

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-250
2. Utility Contact: V. T. Chilson (305) 552-3824
3. Licensed Thermal Power (MWt): 2200
4. Nameplate Rating (Gross MWe): 894 x 0.85 = 760
5. Design Electrical Rating (Net MWe): 693
6. Maximum Dependable Capacity (Gross MWe): 700
7. Maximum Dependable Capacity (Net MWe): 666
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

ITEM 21 CALCULATED BY USING A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	696.0	1,440.0	63,441.6
12. Hours Reactor Was Critical	636.7	636.7	47,906.1
13. Reactor Reserve Shutdown Hours	.0	.0	213.3
14. Hours Generator On-Line	512.2	512.2	46,202.4
15. Unit Reserve Shutdown Hours	.0	.0	121.8
16. Gross Thermal Energy (MWH)	1,033,604	1,033,604	92,502,238
17. Gross Electrical Energy (MWH)	335,060	335,060	29,416,060
18. Net Electrical Energy (MWH)	314,299	312,012	27,834,106
19. Unit Service Factor	73.6	35.6	72.8
20. Unit Availability Factor	73.6	35.6	73.0
21. Unit Capacity Factor (MDC Net)	67.8	32.5	66.4
22. Unit Capacity Factor (DER Net)	65.2	31.3	63.3
23. Unit Forced Outage Rate	4.8	4.8	2.7

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
23	12/01/79	S	133.4	C	4		RC	FUELXX	UNIT NO. 3 WAS REMOVED FROM SERVICE FOR SCHEDULED REFUELING, MAINTENANCE, AND INSPECTIONS. OUTAGE WAS EXTENDED TO REPAIR FAILED MECHANICAL SEAL ON REACTOR COOLANT PUMP NO. 3C. CORRECTIVE ACTIONS INCLUDED REPLACING A SHEARED LOCK PIN AND SHAFT SLEEVE ON SHAFT SEAL ASSEMBLY.
1	02/06/80	F	5.5	B	1		HA	TURBIN	UNIT NO. 3 WAS REMOVED FROM SERVICE TO BALANCE THE TURBINE ROTOR.
2	02/07/80	F	17.9	B	1		HA	TURBIN	UNIT NO. 3 WAS REMOVED FROM SERVICE TO BALANCE THE TURBINE ROTON.
3	02/08/80	S	.8	B	1		HA	TURBIN	UNIT NO. 3 WAS REMOVED FROM SERVICE TO PERFORM PERIODIC TEST ON TURBINE OVERSPEED PROTECTION SYSTEM.
4	02/08/80	F	2.3	A	3		HA	INSTRU	UNIT NO. 3 WAS TRIPPED BY ACTUATION OF LOSS OF EXCITATION RELAY DUE TO A FALSE SIGNAL FROM A MODULE IN THE VOLTAGE REGULATOR. CORRECTIVE ACTIONS INCLUDED ADJUSTING AN OPERATIONAL AMPLIFIER IN THE VOLTAGE REGULATOR CONTROL SYSTEM.
5	02/20/80	S	12.7	B	1		HA	TURBIN	UNIT NO. 3 WAS REMOVED FROM SERVICE TO BALANCE THE TURBINE ROTOR.
6	02/21/80	S	11.2	B	1		HA	TURBIN	UNIT NO. 3 WAS REMOVED FROM SERVICE TO BALANCE THE TURBINE ROTOR.
7	02/25/80	F	.0	A	5		HC	HTEXCH	LOAD REDUCTION DUE TO SECONDARY WATER CHEMISTRY LIMITATIONS.

SUMMARY

OPERATION WAS ROUTINE FOLLOWING RETURN TO POWER ON 02/06/80 FROM A REFUELING AND MAINTENANCE OUTAGE. THE UNIT WAS AGAIN REMOVED FROM SERVICE ON 02/06/80, 02/07/80, 02/20/80, AND 02/21/80 TO BALANCE THE ROTOR, AND ON 02/08/80 FOR PERIODIC TESTING OF THE TURBINE OVERSPEED PROTECTION SYSTEM. A UNIT TRIP ALSO OCCURRED ON 02/08/80 WHEN THE LOSS OF EXCITATION RELAY ACTUATED ON A FALSE SIGNAL IN THE VOLTAGE REGULATOR. ON 2/25/80, LOAD WAS REDUCED DUE TO SECONDARY WATER CHEMISTRY LIMITATIONS.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

TURKEY POINT 4

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Florida/Dade 25 miles S of Miami
2.	Electric Reliability Council	Southeastern
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2200
5.	Design Electrical Rating (MWE Net)	693
6.	Date of Initial Criticality	June 11, 1973
7.	Date of Commercial Operation	September 7, 1973
8.	Condenser Cooling Method	Recirculating Canal System
9.	Condenser Cooling Water Source	Biscayne Bay

Utility & Contractor Information

10.	Licensee	Florida Power and Light Company
11.	Corporate Address	9250 West Flagler Street P. O. Box 013100 Miami, Florida 33101
12.	Corporate Contact	R. E. Uhrig, V. P. Nuclear Affairs
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. Lewis
18.	IE Senior Inspector	E.H. Verdery
19.	IE Resident Inspector	R. J. Vogt-Lowell (305) 245-2910
20.	Licensing Project Manager	M. Grotenhuis
21.	Docket Number	50-251
22.	License Number/Date of Issue	DPR-41, 4/10/73
23.	Public Document Room	Environmental and Urban Affairs Library Florida International University Miami, Florida 33199

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO ETS SECTIONS 5.3.1, 5.3.2 AND 5.1, THE LICENSEE FAILED TO PROVIDE REQUIRED APPROVAL OF WRITTEN PROCEDURES FOR MONITORING, SURVEILLANCE, AND SPECIAL STUDY PROGRAMS DEFINED IN SECTION 4.0 OF THE ETS.
(7939 DEFICIENCY)

CONTRARY TO THE REQUIREMENTS OF 10 CFR 20.201(B)(1), ON DECEMBER 11, 1979 A CONTRACT WORKER MARKING THE TUBE SHEET OF STEAM GENERATOR "B" IN UNIT 3 WAS EXPOSED TO 3.55 REMS WHOLE BODY DOSE.
(7940 INFRACTION)

CONTRARY TO THE REQUIREMENTS OF TECH SPEC 6.8.1, PRIOR TO DECEMBER 11, 1979, NO PROCEDURE HAD BEEN ESTABLISHED FOR THE MARKING OF DEFECTIVE STEAM GENERATOR TUBES FOR SUBSEQUENT TUBE PLUGGING.
(7940 INFRACTION)

CONTRARY TO THE REQUIREMENTS OF TECHNICAL SPECIFICATION 6.8.1, THE REQUIREMENT OF RWP 79-681 WHERE NOT MET IN THAT 10 SEPARATE CHANNEL HEAD ENTRIES WERE MADE WITHOUT H.P. COVERAGE AND NO CHANNEL HEAD AIR SAMPLE WAS TAKEN DURING THE JOB.
(7960 INFRACTION)

AS REQUIRED BY TECHNICAL SPECIFICATION 6.8.1, WRITTEN PROCEDURES AND ADMINISTRATIVE POLICIES SHALL BE IMPLEMENTED. PARAGRAPH 8.4.6.1.1 OF ADMINISTRATIVE PROCEDURE A.P. 103.2 REQUIRES THAT ALL RECORDER CHARTS BE STAMPED WITH THE DATE AND IDENTIFICATION. CONTRARY TO THE ABOVE, RECORDER CHARTS 454, 455 AND 456 ASSOCIATED WITH THE STEAM GENERATOR PARAMETERS GATHERED DURING AND SUBSEQUENT TO THE 1:37 A.M. UNIT 4 REACTOR TRIP ON DECEMBER 19, 1979, WERE NEITHER STAMPED NOR LABELED WITH ANY IDENTIFICATION OR DATES.
(7941 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

* UNIT DUE TO SHUT DOWN FOR STEAM GENERATOR TUBE INSPECTION.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

UNIT OPERATING AT 100% POWER.

* LAST IE SITE INSPECTION DATE: FEBRUARY 28-29, 1980

* INSPECTION REPORT NO: 50-251/80-11

SUMMARY

INSPECTION SUMMARY

* INSPECTION DECEMBER 17-20 (79-40): THIS SPECIAL, UNANNOUNCED INSPECTION INVOLVED 13 1/2 INSPECTOR-HOURS ON SITE IN THE AREA OF REVIEW OF EVENTS SURROUNDING THE OVEREXPOSURE OF A CONTRACT WORKER. THREE APPARENT ITEMS OF NONCOMPLIANCE WERE FOUND IN THE ONE AREA INSPECTED (INFRACTION - EXCEEDED QUARTERLY WHOLE BODY RADIATION EXPOSURE LIMIT; INFRACTION - FAILURE TO HAVE A PROCEDURE; AND INFRACTION - FAILURE TO FOLLOW PROCEDURES).

INSPECTION JANUARY 8-11 (80-01): THIS ROUTINE UNANNOUNCED INSPECTION INVOLVED 10 INSPECTOR-HOURS ON SITE IN THE AREAS OF INSERVICE-INSPECTION DATA REVIEW AND EVALUATION; WELDING AND NONDESTRUCTIVE TESTING; IE BULLETIN 79-13: LP INSPECTION OF PRESSURIZER NOZZLE/SAFE END WELDS. OF THE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
		NONE		

INSPECTION JANUARY 18-21 (80-02): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 7 1/2 INSPECTOR-HOURS ON SITE IN THE AREAS OF ENGINEERED SAFEGUARDS AND EMERGENCY POWER SYSTEMS INTEGRATED TESTING AND PERSONNEL AIRLOCK TESTING. OF THE TWO AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION JANUARY 7-11 (80-03): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 13 1/2 INSPECTOR-HOURS ON SITE IN THE AREA OF UPGRADING OF THE TURKEY POINT FIRE PROTECTIN PROGRAM. IN THE AREA INSPECTED, NO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED, BUT ONE APPARENT DEVIATION WAS FOUND (DEVIATION - FAILURE TO IMPLEMENT A FIRE PROTECTION QUALITY ASSURANCE PROGRAM).

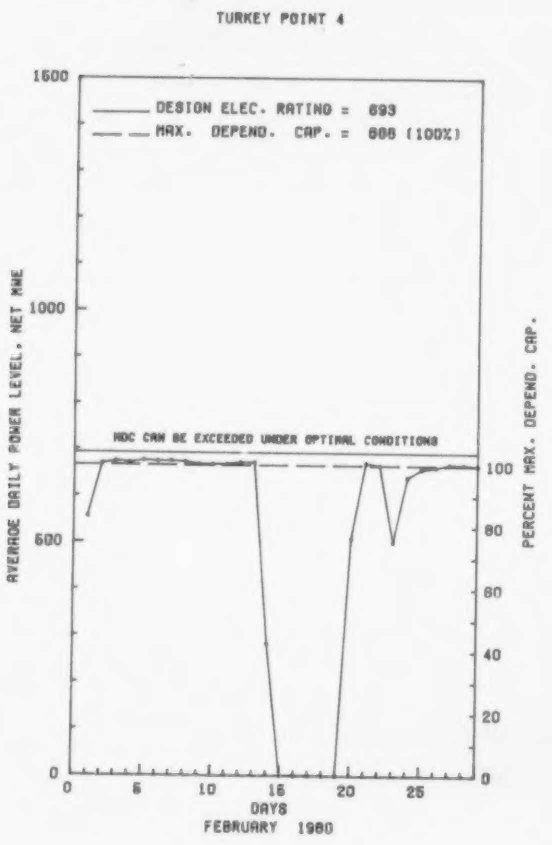
INSPECTION FEBRUARY 5-7 (80-05): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 10 INSPECTOR-HOURS ON SITE IN THE AREAS OF PIPE SUPPORT BASEPLATE DESIGNS USING CONCRETE EXPANSION ANCHOR BOLTS (IEB 79-02); SEISMIC ANALYSIS FOR AS-BUILT SAFETY-RELATED PIPING SYSTEMS (IEB 79-14). OF THE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

REVIEWED BY OMPA LOEB: T. C. Cintula **DATE:** 4·10·80

REVIEWED BY NRR DOR: **DATE:** 4·11·80

TURKEY POINT 4

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-251
2. Utility Contact: V. T. Chilson (305) 552-3824
3. Licensed Thermal Power (MWt): 2200
4. Nameplate Rating (Gross MWe): 894 x 0.85 = 760
5. Design Electrical Rating (Net MWe): 693
6. Maximum Dependable Capacity (Gross MWe): 700
7. Maximum Dependable Capacity (Net MWe): 666
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

ITEM 21 CALCULATED BY USING A WEIGHTED AVERAGE.

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	696.0	1,440.0	57,169.0
12. Hours Reactor Was Critical	569.4	1,299.5	42,127.3
13. Reactor Reserve Shutdown Hours	.0	12.3	166.6
14. Hours Generator On-Line	559.8	1,256.1	40,438.6
15. Unit Reserve Shutdown Hours	.0	12.3	31.2
16. Gross Thermal Energy (MWH)	1,206,216	2,706,064	83,925,044
17. Gross Electrical Energy (MWH)	384,015	861,585	26,765,063
18. Net Electrical Energy (MWH)	363,058	816,502	25,346,495
19. Unit Service Factor	80.4	87.2	70.7
20. Unit Availability Factor	80.4	88.1	70.8
*21. Unit Capacity Factor (MDC Net)	78.3	85.1	67.0
22. Unit Capacity Factor (DER Net)	75.3	81.8	64.0
23. Unit Forced Outage Rate	.2	.1	3.2

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
MAINTENANCE & INSPECTIONS - MARCH 24 - MAY 14, 1980
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
5	02/01/80	F	.0	A	5		HC	HTEXCH	LOAD REDUCTION TO REPAIR CONDENSER TUBE LEAK.
6	02/14/80	S	134.8	A	3		HA	MECFUN	UNIT WAS REMOVED FROM SERVICE TO REPAIR TURBINE CONTROL OIL SYSTEM. CORRECTIVE ACTIONS INCLUDED REPLACING TURBINE GOVERNOR IMPELLER SHAFT SLEEVE.
7	02/20/80	F	1.4	A	3		HA	MECFUN	UNIT NO. 4 WAS TRIPPED BY THE TURBINE PROTECTION SYSTEM DUE TO A FALSE SIGNAL FROM THE LOW VACUUM TRIP DEVICE. CORRECTIVE ACTIONS INCLUDED REPAIRING A LEAK AT A FITTING ON THE VACUUM SENSING LINE.
8	02/23/80	F	.0	A	5		HC	HTEXCH	LOAD REDUCTION TO REPAIR CONDENSER TUBE LEAK.

SUMMARY

OPERATION WAS ROUTINE DURING THE REPORT PERIOD, EXCEPT FOR A UNIT TRIP BY THE TURBINE PROTECTION SYSTEM ON 02/20/80. THE UNIT WAS TAKEN OFF-LINE ON 02/14/80 TO REPAIR THE TURBINE CONTROL OIL SYSTEM. LOAD REDUCTIONS OCCURRED ON 02/01/80 AND 02/23/80 TO REPAIR CONDENSER TUBE LEAKS.

LEGEND

[1] Type:
F Forced
S Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C Refueling
D Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

VERMONT YANKEE

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Vermont/Windham 5 miles S of Brattleboro
2.	Electric Reliability Council	NPCC
3.	Type of Reactor	BWR
4.	Power Level (MWT)	1593
5.	Design Electrical Rating (MWE Net)	514
6.	Date of Initial Criticality	March 24, 1972
7.	Date of Commercial Operation	November 30, 1972
8.	Condenser Cooling Method	Closed Cycle Cooling Towers
9.	Condenser Cooling Water Source	Connecticut River

Utility & Contractor Information

10.	Licensee	Vermont Yankee Nuclear Power Corp.
11.	Corporate Address	20 Turnpike Road Westboro, Massachusetts 01581
12.	Corporate Contact	D. W. Edwards Assistant to the Vice President
13.	Architect/Engineer	Ebasco
14.	Nuclear Steam Supply System	GE
15.	Constructor	Ebasco

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	IE Senior Inspector	E. McCabe
19.	IE Principal Inspector	S. Collins
20.	Licensing Project Manager	V. L. Rooney
21.	Docket Number	50-271
22.	License Number/Date of Issue	DPR-28, 3/21/72
23.	Public Document Room	Brooks Memorial Library 224 Main Street Brattleboro, Vermont 05301

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS:

- Routine operations at full power. SJAE offgas release rate is 3190 microcuries per second. Stack release rate is below 100 microcuries per second.

LAST IE SITE INSPECTION DATE: 2/8/80

INSPECTION REPORT NO: 50-271/80-02

SUMMARY

INSPECTION SUMMARY

- No reports issued.

REPORTS RECEIVED FROM LICENSEE

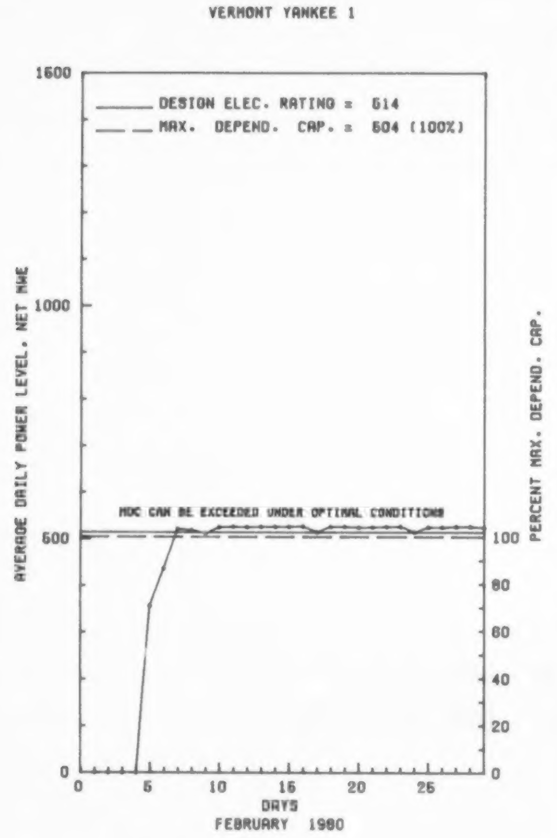
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
80-001/03L	01/02/80	01/29/80	30 Day LER	LPCI/core spray low pressure permissive pressure switch (PS2-3-52C) trip setting found below TS allowable value during surveillance
80-003/03L	01/07/80	02/04/80	30 Day LER	Environmental air sampling at one station missed during period from 12/30 to 1/7 due to inoperable sample pump
80-004/03L	01/09/80	02/04/80	30 Day LER	Cooling fan for rod position indication system power supply failed causing intermittent loss of rod position indication for about 2 hours
80-005/03L	01/15/80	02/14/80	30 Day LER	Quarterly environmental surveillance data collected beyond prescribed Tech Spec interval
80-006/01T	01/22/80	02/04/80	14 Day LER	Both SJAE offgas radiation monitors inoperable for 20 minutes due to flow blockage in sample lines

REVIEWED BY OMPA LOEB: John J. Pozzo DATE: 4-10-80

REVIEWED BY NRR DOR: DATE: 4/11/80

VERMONT YANKEE

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-271
2. Utility Contact: F.C. BEERS (617) 366-9011
3. Licensed Thermal Power (MWt): 1593
4. Nameplate Rating (Gross MWe): 626 x 0.9 = 563
5. Design Electrical Rating (Net MWe): 514
6. Maximum Dependable Capacity (Gross MWe): 535
7. Maximum Dependable Capacity (Net MWe): 504
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	696.0	1,440.0	65,234.8
12 Hours Reactor Was Critical	611.7	1,334.7	52,493.3
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	600.5	1,310.6	50,638.4
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	934,568	1,965,965	71,190,576
17. Gross Electrical Energy (MWH)	320,755	672,576	23,642,246
18. Net Electrical Energy (MWH)	307,911	645,451	22,413,816
19. Unit Service Factor	86.3	91.0	77.6
20. Unit Availability Factor	86.3	91.0	77.6
21. Unit Capacity Factor (MDC Net)	87.8	88.9	68.2
22. Unit Capacity Factor (DER Net)	86.1	87.2	66.8
23. Unit Forced Outage Rate	.0	2.0	6.4

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-6	01/31/80	S	95.5	D	4		ZZ	ZZZZZZ	SHUTDOWN FOR MODIFICATIONS AS REQUIRED BY NUREG 0578.
80-7	02/04/80	S	.0	H	5		RA	CONROD	TURBINE SURVEILLANCE AND CONTROL ROD PATTERN ADJUSTMENT.
80-8	02/24/80	S	.0	H	5		RA	CONROD	TURBINE SURVEILLANCE AND CONTROL ROD PATTERN ADJUSTMENT.

SUMMARY

OPERATION WAS AT NEAR FULL POWER FOLLOWING RETURN TO SERVICE ON 02/04/80 FROM AN OUTAGE TO PERFORM TMI-RELATED MODIFICATIONS. LOAD REDUCTIONS OCCURRED ON 02/04/80 AND 02/24/80 FOR TURBINE SURVEILLANCE AND CONTROL ROD PATTERN ADJUSTMENT.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

YANKEE ROWE

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Massachusetts/Franklin 25 miles NE of Pittsfield
2.	Electric Reliability Council	NPCC
3.	Type of Reactor	PWR
4.	Power Level (MWT)	600
5.	Design Electrical Rating (MWE Net)	175
6.	Date of Initial Criticality	August 19, 1960
7.	Date of Commercial Operation	July 1961
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Deerfield River

Utility & Contractor Information

10.	Licensee	Yankee Atomic Electric Company
11.	Corporate Address	20 Turnpike Road Westboro, Massachusetts 01581
12.	Corporate Contact	D. W. Edwards, Asst. to V.P.
13.	Architect/Engineer	Stone & Webster
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Senior Inspector	R. R. Keimig
19.	IE Principal Inspector	W. J. Lazarus
20.	Licensing Project Manager	A. Burger
21.	Docket Number	50-29
22.	License Number/Date of Issue	DPR-3, 12/24/63
23.	Public Document Room	Greenfield Community College 1 College Drive Greenfield, Massachusetts 01301

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

```
    I N S P E C T I O N   S T A T U S

    OTHER ITEMS OF INTEREST

    SYSTEMS AND COMPONENT PROBLEMS:

+   The LP Turbine has failed (thrown buckets,
    bent casing, etc.).

    FACILITY ITEMS (PLANS AND PROCEDURES):

+   Length of shutdown has not been determined.

    PLANT STATUS:

+   Shutdown and cooled down.

+ LAST IE SITE INSPECTION DATE:  3/7/80

+        INSPECTION REPORT NO:  50-29/80-06

    SUMMARY

    INSPECTION SUMMARY

+   No reports issued.
```

REPORTS RECEIVED FROM LICENSEE

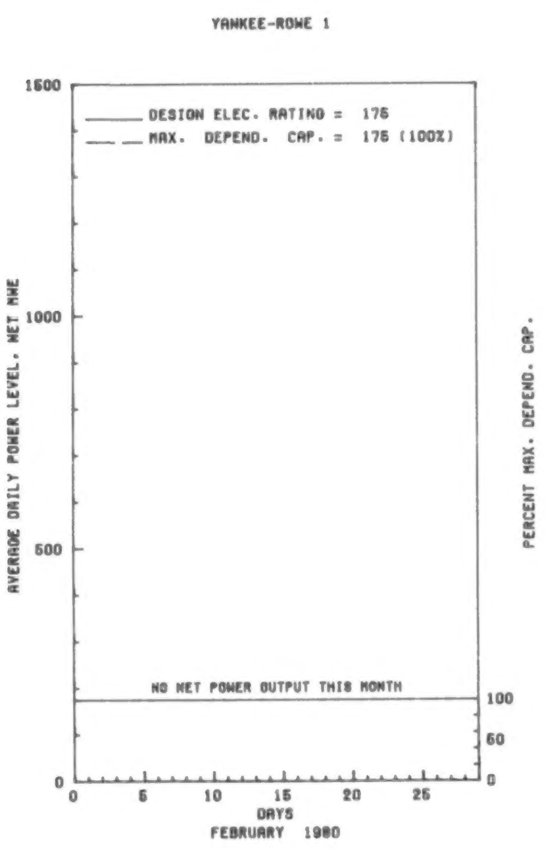
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-033/ 03L	12/31/79	01/30/80	30 Day LER	Flow path to BAMT failed operability check
80-001/ 03L	01/03/80	02/01/80	30 Day LER	Steam generator blowdown monitor out of service
80-002/ 03L	01/16/80	02/15/80	30 Day LER	Primary vent stack iodine channel failure
80-003/ 03L	01/18/80	02/15/80	30 Day LER	Missed sealed source surveillance
80-004/ 03L	01/19/80	02/15/80	30 Day LER	Secondary water chlorides out of specification
80-008/ 01T	02/01/80	02/15/80	24 Hour LER	Inoperable nitrogen accumulator relays

REVIEWED BY OMPA LOEB: T.C. Cintula **DATE:** 4·10·80

REVIEWED BY NRR DOR: L. Burger **DATE:** 4-14-80

YANKEE ROWE

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-029
2. Utility Contact: F.C. BEERS (617) 366-9011
3. Licensed Thermal Power (MWt): 600
4. Nameplate Rating (Gross MWe): 185 x 1.0 = 185
5. Design Electrical Rating (Net MWe): 175
6. Maximum Dependable Capacity (Gross MWe): 180
7. Maximum Dependable Capacity (Net MWe): 175
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

CAPACITY FACTORS CALCULATED WITH A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	696.0	1,440.0	169,077.0
12. Hours Reactor Was Critical	6.1	445.8	138,810.4
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	.0	439.7	134,467.1
15. Unit Reserve Shutdown Hours	.0	.0	0
16. Gross Thermal Energy (MWH)	0	253,769	72,002,091
17. Gross Electrical Energy (MWH)	0	80,635	22,152,788
18. Net Electrical Energy (MWH)	0	75,749	20,741,893
19. Unit Service Factor	.0	30.5	79.5
20. Unit Availability Factor	.0	30.5	79.5
21. Unit Capacity Factor (MDC Net)	.0	30.1	72.4
22. Unit Capacity Factor (DER Net)	.0	30.1	72.4
23. Unit Forced Outage Rate	100.0	61.3	2.1

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: UNKNOWN

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	01/19/80	F	696.0	D	4				PLANT WAS SHUTDOWN TO INSTALL VOLTAGE REGULATORS AND MAKE TMI RELATED CHANGES UNTIL 02/05/80. REMAINED SHUTDOWN UNTIL 02/12/80 TO REPAIR MAIN COOLANT PUMP FLANGE LEAK. REMAINED SHUTDOWN THE REST OF THE MONTH DUE TO TURBINE ROTOR FAILURE.

SUMMARY

THE UNIT REMAINED SHUTDOWN THROUGHOUT THE REPORT PERIOD FOR INSTALLATION OF VOLTAGE REGULATORS AND FOR TMI-RELATED MODIFICATIONS. THE OUTAGE WAS EXTENDED TO REPAIR A FLANGE LEAK IN THE MAIN COOLANT PUMP, AND BECAUSE OF TURBINE ROTOR FAILURE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

ZION 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Illinois/Lake 40 miles N of Chicago
2.	Electric Reliability Council	Main
3.	Type of Reactor	PWR
4.	Power Level (MWT)	3250
5.	Design Electrical Rating (MWE Net)	1040
6.	Date of Initial Criticality	June 19, 1973
7.	Date of Commercial Operation	December 31, 1973
8.	Condenser Cooling Method	Straight Through
9.	Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10.	Licensee	Commonwealth Edison Co.
11.	Corporate Address	P. O. Box 767 Chicago, Illinois
12.	Corporate Contact	D. L. Peoples
13.	Architect/Engineer	Sargent & Lundy
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Various

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Senior Inspector	R. Spessard
19.	IE Resident Inspector	J. E. Kohler (312) 746-2313
20.	Licensing Project Manager	E. Reeves
21.	Docket Number	50-295
22.	License Number/Date of Issue	DPR-39 issued 4/6/73
23.	Public Document Room	Zion-Benton Public Library 2600 Emmans Avenue Zion, Illinois 60099

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

+ Feedwater piping removed at the steam generator nozzles and replaced on both Units 1 and 2. (79-22)

PLANT STATUS

+ Unit 1 started up February 19, 1980 and is now operating routinely.

+LAST IE SITE INSPECTION DATE: November 29; December 20, 1979 and
+ January 3-4 and 10, 1980

+ INSPECTION REPORT NO: 79-26

SUMMARY

INSPECTION SUMMARY

+ Inspections on September 10; October 15-17, 19, 27, 29, 31; November 1-2, 6-9, 14-15, 20-21, 26-29; December 4-7, 11-14, 17-21, 26-28, 1979 and January 2-4, 1980, (79-22): Review of ISI program, procedures, personnel and equipment qualifications, observation of work activities, and review of NDE examination documentation; followup of licensee activities related to IE Bulletin 79-13, Cracking in Feedwater Piping System, followup of licensee activities related to IE Bulletin 79-17, Pipe Cracking in Stagnant Borated Water Systems at Power Plants. The inspection involved 323 inspector hours onsite by one NRC inspector. No items of noncompliance or deviations were identified.

+ Inspection on November 29; December 20, 1979; January 3-4 and 10, 1980, (79-26): Special inspection of damaged Bergen-Paterson large bore hydraulic snubbers installed on Unit 1 steam generators. The inspection involved 28 inspector hours onsite by one NRC inspector. No items of noncompliance or deviations were identified.

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
80-02/ 03L-0	12/30/79	01/30/80	30 Day LER	While both units were in cold shutdown the fire sump pump radiation monitor (ORT-PR25) was bypassed.
80-05/ 03L-0	01/10/80	02/07/80	30 Day LER	While in cold shutdown the misc. vent gas rad monitor ORT-PR18A failed low.
80-08/ 01T-0	12/04/79	02/19/80	14 Day LER	While in CSD Unit 1 RCS was diluted with containment integrity violated.

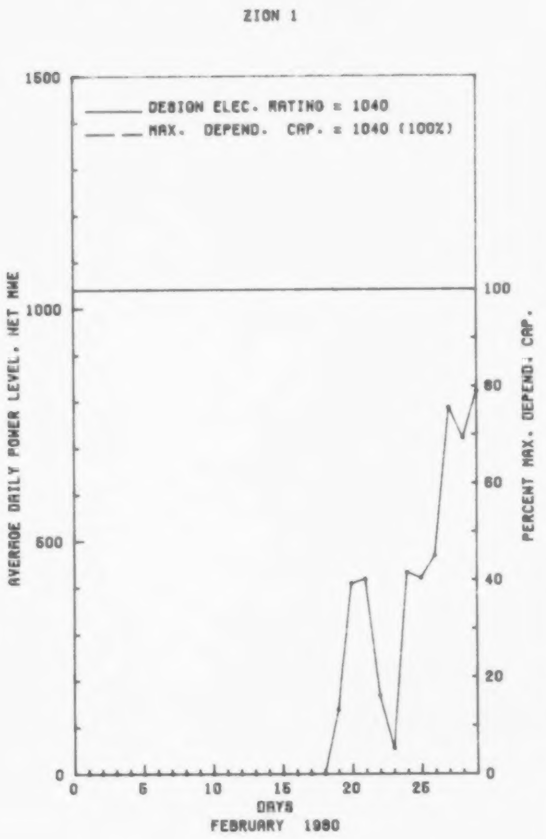
==

REVIEWED BY OMPA LOEB: T.C. Cintula **DATE:** 4-10-80

REVIEWED BY NRR DOR: Ed A. Reeves **DATE:** 4-14-80

ZION 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-295
2. Utility Contact: J. COOK (312) 746-2084
3. Licensed Thermal Power (MWt): 3250
4. Nameplate Rating (Gross MWe): 1220 x 0.9 = 1098
5. Design Electrical Rating (Net MWe): 1040
6. Maximum Dependable Capacity (Gross MWe): 1085
7. Maximum Dependable Capacity (Net MWe): 1040
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	696.0	1,440.0	54,048.0
12. Hours Reactor Was Critical	316.7	316.7	37,424.6
13. Reactor Reserve Shutdown Hours	.0	.0	2,621.8
14. Hours Generator On-Line	224.9	224.9	36,312.2
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	425,855	426,855	99,809,856
17. Gross Electrical Energy (MWH)	126,905	126,905	32,758,955
18. Net Electrical Energy (MWH)	101,241	101,241	30,585,487
19. Unit Service Factor	32.3	15.6	67.2
20. Unit Availability Factor	32.3	15.6	67.2
21. Unit Capacity Factor (MDC Net)	14.0	6.8	54.4
22. Unit Capacity Factor (DER Net)	14.0	6.8	54.4
23. Unit Forced Outage Rate	67.7	84.4	15.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type^1	Duration (Hours)	Reason^2	Method of Shutting Down Reactor^3	Licensee Event Report No.	System Code^4	Component Code^5	Cause & Corrective Action to Prevent Recurrence
3	01/24/80	F	168.0	A	4				REPLACE 1B RCP SEALS WHICH REQUIRED REPAIR AND EXTENDED THE OUTAGE.
4	02/08/80	F	275.5	A	9				REPAIR OF CHARGING PUMP ISOLATION VALVES WHICH EXTENDED THE OUTAGE.
5	02/22/80	F	8.5	A	3				LOSS OF 1B FWP.
6	02/22/80	F	15.2	A	3				REACTOR TRIP CAUSED BY A GOVERNOR VALVE OPENING.
7	02/23/80	F	3.9	A	1				REPAIR OF 1A MSIV DC SOLENOID.

SUMMARY

THE UNIT RETURNED TO POWER ON 02/19/80 FOLLOWING REPAIR OF THE 1B REACTOR COOLANT PUMP SEALS AND CHARGING PUMP ISOLATION VALVES. ON 02/22/80, A REACTOR TRIP OCCURRED ON LOSS OF THE 1B FEEDWATER PUMP, AND THE UNIT TRIPPED DUE TO A GOVERNOR VALVE OPENING. THE UNIT WAS TAKEN OFF-LINE BRIEFLY ON 02/23/80 FOR REPAIR TO THE 1A MAIN STEAM ISOLATION VALVE SOLENOID

LEGEND

^{1}Type:
- F--Forced
- S--Scheduled

^{2}Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

^{3}Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

^{4}Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

^{5}Exhibit H: Same source.

ZION 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Illinois/Lake 40 miles N of Chicago
2.	Electric Reliability Council	Main
3.	Type of Reactor	PWR
4.	Licenses Power Level (MWT)	3250
5.	Design Electrical Rating (MWE Net)	1040
6.	Date of Initial Criticality	December 24, 1973
7.	Date of Commercial Operation	September 17, 1974
8.	Condenser Cooling Method	Straight Through
9.	Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10.	Licensee	Commonwealth Edison Co.
11.	Corporate Address	P. O. Box 767 Chicago, Illinois
12.	Corporate Contact	D. L. Peoples, Manager of Nuclear Licensing
13.	Architect/Engineer	Sargent & Lundy
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Various

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Senior Inspector	R. Spessard
19.	IE Resident Inspector	J. E. Kohler (312) 746-2313
20.	Licensing Project Manager	E. Reeves
21.	Docket Number	50-304
22.	License Number/Date of Issue	DPR-48, 11/14/73
23.	Public Document Room	Zion-Benton Public Library 2600 Emmans Avenue Zion, Illinois 60099

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

+ Feedwater piping removed at the steam generator nozzles and replaced on both Units 1 and 2. (79-24)

PLANT STATUS

+ Unit operating routinely.

+LAST IE SITE INSPECTION DATE: November 29; December 20, 1979 and January 3-4 and 10, 1980

+ INSPECTION REPORT NO: 79-25

SUMMARY

INSPECTION SUMMARY

+ Inspections on September 10; October 15-17, 19, 27, 29, 31; November 1-2, 6-9, 14-15, 20-21, 26-29; December 4-7, 11-14, 17-21, 26-28, 1979 and January 2-4, 1980, (79-24): Followup of licensee activities related to IE Bulletin 79-13, Cracking in Feedwater Piping System, followup of licensee activities related to IE Bulletin 79-17, Pipe Cracking in Stagnant Borated Water Systems at Power Plants. The inspection involved 323 inspector hours onsite by one NRC inspector. No items of noncompliance or deviations were identified.

+ Inspection on November 29; December 20, 1979; January 3-4 and 10, 1980, (79-25): Special inspection of damaged Bergen-Paterson large bore hydraulic snubbers installed on Unit 1 steam generators. The inspection involved 28 inspector hours onsite by one NRC inspector. No items of noncompliance or deviations were identified.

REPORTS RECEIVED FROM LICENSEE

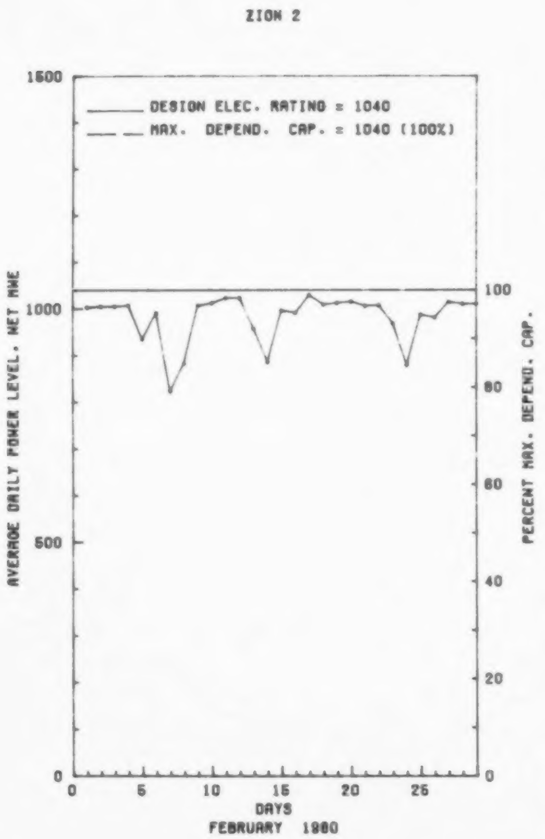
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
80-01/03L-0	01/13/80	02/11/80	30 Day LER	While in cold shutdown, power supplies for the blowers for the radiation monitors on a non-essential bus were de-energized.

REVIEWED BY OMPA LOEB: T.C. Cintula **DATE:** 4·10·80

REVIEWED BY NRR DOR: Ed A. Reeves **DATE:** 4-14-80

ZION 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-304
2. Utility Contact: J. COOK (312) 746-2084
3. Licensed Thermal Power (MWt): 3250
4. Nameplate Rating (Gross MWe): 1220 x 0.9 = 1098
5. Design Electrical Rating (Net MWe): 1040
6. Maximum Dependable Capacity (Gross MWe): 1085
7. Maximum Dependable Capacity (Net MWe): 1040
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	696.0	1,440.0	47,761.0
12. Hours Reactor Was Critical	696.0	980.6	34,584.2
13. Reactor Reserve Shutdown Hours	.0	.0	226.1
14. Hours Generator On-Line	696.0	964.1	33,778.0
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	2,179,482	2,974,854	95,610,172
17. Gross Electrical Energy (MWH)	714,607	972,370	30,693,590
18. Net Electrical Energy (MWH)	683,357	923,162	29,124,628
19. Unit Service Factor	100.0	67.0	70.7
20. Unit Availability Factor	100.0	67.0	70.7
21. Unit Capacity Factor (MDC Net)	94.4	61.6	58.6
22. Unit Capacity Factor (DER Net)	94.4	61.6	58.6
23. Unit Forced Outage Rate	.0	33.0	18.2

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUELING - MAY 3, 1980 - 7 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	NONE								

SUMMARY

OPERATION WAS AT NEAR FULL POWER DURING THE REPORT PERIOD, WITH NO SHUTDOWNS OR OUTAGES REPORTED.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

SECTION 3 — APPENDIX

STATUS OF SPENT FUEL STORAGE CAPABILITY

DATA AS OF 02-29-80

Facility	Core Size[a] (No. of Assemblies)	Present Authorized Storage Pool Capacity (Fuel Assemblies)	No. of Assemblies Stored	Remaining Capacity (No. of Assemblies)	Remaining Capacity If Pending Request Approved (No. of Assemblies)	Scheduled Date For Next Refueling	Will Fill Present[b] Authorized Capacity
I. Pressurized Water Reactors							
ARKANSAS 1	177	590	176	414		12-80	1986
ARKANSAS 2 (d)	177	480	0	480		03-81	1993
BEAVER VALLEY 1	157	833	0	833		N/S	1995
CALVERT CLIFFS 1	217	1056 (c)	364 (c)	692 (c)		10-80	1984
CALVERT CLIFFS 2	217					01-81	
COOK 1	193	500	193	307	1857	N/S	1985
COOK 2	193					N/S	
CRYSTAL RIVER 3	177	256	60	196	1105	N/S	1982
DAVIS-BESSE 1	177	735	0	735		04-80	1989
FARLEY 1	157	675	0	675		N/S	1990
FORT CALHOUN 1	133	483	197	286		05-81	1985
GINNA	121	595	136	459		03-80	1987
HADDAM NECK	157	1168	343	825		05-80	1994
INDIAN POINT 1	0	288	160	128		N/S	
INDIAN POINT 2	193	482	200	282		N/S	1984
INDIAN POINT 3	193	837	64	773		N/S	1989
KEWAUNEE	121	990	120	870		05-80	1999
MAINE YANKEE	217	953	433	520		N/S	1986
MILLSTONE 2	217	667	144	523		07-80	1987
NORTH ANNA 1	157	400	0	400	966	N/S	1984
NORTH ANNA 2	157	400	0	400		N/S	
OCONEE 1	177	1224 (l)	545	(l)	675	02-81	1986
OCONEE 2	177					03-80	
OCONEE 3	177					12-80	
PALISADES	204	798	273	525		N/S	1985
POINT BEACH 1	121	1502 (c)	212 (c)	1290 (c)		N/S	1995
POINT BEACH 2	121					04-80	1995
PRAIRIE ISLAND 1	121	687 (c)	280 (c)	407 (c)		06-80	1984
PRAIRIE ISLAND 2	121					N/S	1984
RANCHO SECO 1	177	579	151	428		07-81	1987
ROBINSON 2	157	272	106	166 (g)		03-80	1980 (g)
SALEM 1	193	264	40	224	1170	03-80	1982
SAN ONOFRE 1	157	216	55	161		04-80	1983
ST LUCIE 1	217	728	60	668		03-80	1990
SURRY 1	157	1044 (c)	1933 (c)	[illegible] (c)		06-80	1983
SURRY 2	157					N/S	
THREE MILE ISLAND 1	177	752	208	544		N/S	1989
THREE MILE ISLAND 2	177	442	0	442		N/S	1986
TROJAN	193	651	64	587		04-80	1986
TURKEY POINT 3	157	821	546 (c)	275 (c)		N/S	1987
TURKEY POINT 4	157					N/S	1987
YANKEE-ROWE 1	76	391	149	242	572	N/S	1985
ZION 1	193	868 (c)	436 (c)	432 (c)	1676	01-81	1983
ZION 2	193					06-80	1983

Facility	Core Size[a] (No. of Assemblies)	Present Authorized Storage Pool Capacity (Fuel Assemblies)	No. of Assemblies Stored	Remaining Capacity (No. of Assemblies)	Remaining Capacity If Pending Request Approved (No. of Assemblies)	Scheduled Date For Next Refueling	Will Fill Present[b] Authorized Capacity
II. Boiling Water Reactors							
BIG ROCK POINT 1	84	193	88	105	353	10-80	1983
BROWNS FERRY 1	764	3471	676	2795		N/S	1985
BROWNS FERRY 2	764	1080	400	680		09-80	1985
BROWNS FERRY 3	764	1528	354	1174		10-80	1985
BRUNSWICK 1	560	(f)	154(PWR)+320(BWR)			04-80	1986
BRUNSWICK 2	560	(f)	91(PWR)+132(BWR)			03-80	1986
COOPER STATION	548	2366	284	2082		03-80	1995
DRESDEN 1	464	672	221	451		N/S	1984
DRESDEN 2	724	2840 (c)	1069 (c)	1771 (c)	6491 (c)	N/S	1985
DRESDEN 3	724					N/S	1984
DUANE ARNOLD	368	2050	364	1686		04-81	1998
FITZPATRICK	560	760	268	492	1732	04-80	1981
HATCH 1	560	840	260	580		N/S	1988
HATCH 2	560	1120	0	1120		N/S	
HUMBOLDT BAY	172	487	251	236		N/S	
LA CROSSE	72	134	113	21	327	04-80	1983
MILLSTONE 1	580	2184	776	1408		09-80	1991
MONTICELLO	484	2237	728	1509		N/S	1990
NINE MILE POINT 1	532	1984	660	1324	2349	N/S	1990
OYSTER CREEK 1	560	1800	620	1180		N/S	1987
PEACH BOTTOM 2	764	2816	618	2198		03-80	1990
PEACH BOTTOM 3	764	2816	712	2104		03-81	1991
PILGRIM 1	580	2320	580	1740		N/S	1990
QUAD CITIES 1	724	1460	596	864		09-80	1986
QUAD CITIES 2	724	1460	492	968		N/S	1985
VERMONT YANKEE 1	368	2000	894	1106		N/S	1986
III. Independent Spent Fuel Storage Installations (h)							
Morris Operations		750 MTU (j)	310	390 MTU (j)	1490 MTU (j)		
NFS(i)		250 MTU	170 MTU	80 MTU			

(a) At each refueling outage approximately 1/3 of a PWR core and 1/4 of a BWR core is off-loaded.
(b) Some of these dates have been adjusted by staff assumptions.
(c) This is the total for both units.
(d) Plant not in commercial operation.
(e) Some spent fuel stored at Brunswick.
(f) Authorized a total 2088 BWR and 304 PWR assemblies for both pools.
(g) Robinson 2 assemblies being shipped to Brunswick for storage.
(h) Capacity is in metric tons of uranium; 1 MTU = 2 PWR assemblies or 5 BWR assemblies.
(i) No longer accepting spent fuel.
(j) Racked for 700 MTU.
(k) Reserved
(l) This is the station total.

N/S = Not Scheduled

REACTOR-YEARS OF EXPERIENCE

AS OF 2/29/80

The total of Reactor years of experience is as the sum of all calender days for each unit, from the date that electricity was first generated until a final shutdown date or the status date, whichever comes first, divided by 365.25 days/year.

If a date is unknown, the first day of the first month of operation is substituted. Units which have not yet generated electricity but which are licensed are listed but not included in the computation.

(INFORMATION INCLUDES BOTH LICENSED AND NON–LICENSED ELECTRICAL UNITS).

LICENSED OPERATING ELECTRICAL PRODUCING UNITS

UNIT NAME	FIRST ELEC. GENERATION	YEARS EXPERIENCE
ARKANSAS 1	08/01/74	5.58
ARKANSAS 2	12/26/78	1.18
BEAVER VALLEY 1	06/14/76	3.71
BIG ROCK POINT 1	12/08/62	17.23
BROWNS FERRY 1	10/15/73	6.38
BROWNS FERRY 2	08/28/74	5.51
BROWNS FERRY 3	09/12/76	3.47
BRUNSWICK 1	12/04/76	3.24
BRUNSWICK 2	04/29/75	4.84
CALVERT CLIFFS 1	01/03/75	5.16
CALVERT CLIFFS 2	12/07/76	3.23
COOK 1	02/10/75	5.05
COOK 2	03/22/78	1.94
COOPER STATION	05/10/74	5.81
CRYSTAL RIVER 3	01/30/77	3.08
DAVIS-BESSE 1	08/28/77	2.51
DRESDEN 1	04/15/60	19.88
DRESDEN 2	04/13/70	9.88
DRESDEN 3	07/22/71	8.61
DUANE ARNOLD	05/19/74	5.79
FARLEY 1	08/18/77	2.54
FITZPATRICK	02/01/75	5.08
FORT CALHOUN 1	08/25/73	6.52
FORT ST VRAIN	12/11/76	3.22
GINNA	12/02/69	10.24
HADDAM NECK	08/07/67	12.57
HATCH 1	11/11/74	5.30
HATCH 2	09/22/78	1.44
INDIAN POINT 2	06/26/73	6.68
INDIAN POINT 3	04/27/76	3.84
KEWAUNEE	04/08/74	5.90
LA CROSSE	04/26/68	11.85
MAINE YANKEE	11/08/72	7.31
MILLSTONE 1	11/29/70	9.25
MILLSTONE 2	11/09/75	4.31
MONTICELLO	03/05/71	8.99
NINE MILE POINT 1	11/09/69	10.31
NORTH ANNA 1	04/17/78	1.87
OCONEE 1	05/06/73	6.82
OCONEE 2	12/05/73	6.24
OCONEE 3	09/01/74	5.50
OYSTER CREEK 1	09/23/69	10.44
PALISADES	12/31/71	8.17
PEACH BOTTOM 2	02/18/74	6.03
PEACH BOTTOM 3	09/01/74	5.50
PILGRIM 1	07/19/72	7.62
POINT BEACH 1	11/06/70	9.32
POINT BEACH 2	08/02/72	7.58
PRAIRIE ISLAND 1	12/04/73	6.24
PRAIRIE ISLAND 2	12/21/74	5.19
QUAD CITIES 1	04/12/72	7.88
QUAD CITIES 2	05/23/72	7.77
RANCHO SECO 1	10/13/74	5.38
ROBINSON 2	09/26/70	9.43
SALEM 1	12/25/76	3.18

LICENSED OPERATING ELECTRICAL PRODUCING UNITS - CONT.

UNIT NAME	FIRST ELEC. GENERATION	YEARS EXPERIENCE
SAN ONOFRE 1	07/16/67	12.63
ST LUCIE 1	05/07/76	3.82
SURRY 1	07/04/72	7.66
SURRY 2	03/10/73	6.98
THREE MILE ISLAND 1	06/19/74	5.70
TROJAN	12/23/75	4.19
TURKEY POINT 3	11/02/72	7.33
TURKEY POINT 4	06/21/73	6.69
VERMONT YANKEE 1	09/20/72	7.44
YANKEE-ROWE 1	11/10/60	19.30
ZION 1	06/28/73	6.67
ZION 2	12/26/73	6.18
TOTAL		452.17 YEARS

PERMANENTLY OR INDEFINITELY SHUTDOWN UNITS

UNIT NAME	FIRST ELEC. GENERATION	SHUTDOWN OR ELECTRICAL CESSATION DATE	YEARS EXPERIENCE
BONUS	08/14/64	06/01/68	3.80
CVTR	12/18/63	01/01/67	3.04
ELK RIVER	08/24/63	02/01/68	4.44
FERMI 1	08/05/66	11/29/72	6.32
HALLAM	05/29/63	09/01/64	1.26
HUMBOLDT BAY	04/18/63	07/02/76	13.21
INDIAN POINT 1	09/16/62	10/31/74	12.12
PATHFINDER	07/25/66	10/01/67	1.19
PEACH BOTTOM 1	01/27/67	11/01/74	7.76
PIQUA	11/04/63	01/01/66	2.16
THREE MILE ISLAND 2	04/21/78	03/28/79	.93
TOTAL			56.23 YEARS

NON–POWER REACTORS IN THE U.S.

1. RESEARCH REACTORS

State	Location	Licensee	Reactor Type	Docket	License Number	Date Issued	Authorized Power Level (KW)
Alabama	1. Tuskegee	Tuskegee Institute	AGN 201 No. 102	50-406	R-122	08-30-74	.0001
Arizona	2. Tucson	U. of Arizona	TRIGA Mk. I	50-113	R-52	12-05-58	100
California	3. Los Angeles	U. of California	Argonaut	50-142	R-71	10-03-60	100
	4. Berkeley	U. of California	TRIGA Mark III	50-224	R-101	08-10-66	1000
	5. Irvine	U. of California	TRIGA Mark I	50-326	R-116	. 11-24-69	250
	6. Santa Barbara	U. of California	L-77	50-433	R-124	12-03-74	.01
	7. Canoga Park	Rockwell International Corporation	L-77	50-94	R-40	05-17-58	.01
	8. Hawthorne	Northrop Corporate Laboratories	TRIGA Mark F	50-187	R-90	03-04-63	1000
	9. San Luis Obispo	California State Polytechnic College	AGN 201 No. 100	50-394	R-121	05-16-73	.0001
	10. Alameda County	General Electric Co.	NTR	50-73	R-33	10-31-57	100
	11. San Diego	General Atomic Co.	TRIGA Mark I	50-89	R-38	05-03-58	250
	12. San Diego	General Atomic Co.	TRIGA Mark F	50-163	R-67	07-01-60	1500
	13. San Ramon	Aerotest Operations, Inc.	TRIGA (Industrial)	50-228	R-98	07-02-65	250
Colorado	14. Jefferson County	U.S. Geological Survey. Dept. of Interior	TRIGA Mark I	50-274	R-113	02-24-69	1000
Delaware	15. Newark	U. of Delaware	AGN-201 No. 113	50-98	R-43	07-03-58	.0001
District of Columbia	16.	Catholic U.	AGN-201 No. 101	50-77	R-31	11-15-67	.0001
Florida	17. Gainesville	U. of Florida	Argonaut	50-83	R-56	05-21-59	100
Georgia	18. Atlanta	Georgia Institute of Technology	AGN-201 No. 104	50-276	R-111	04-19-68	.0001
	19. Atlanta	Georgia Institute of Technology	Heavy Water	50-160	R-97	12-29-64	5000
Idaho	20. Pocatello	Idaho State U.	AGN-201 No. 103	50-284	R-110	10-11-67	.0001
Illinois	21. Urbana	U. of Illinois	Advanced TRIGA	50-151	R-115	07-22-69	1500
	22. Urbana	U. of Illinois	LOPRA	50-356	R-117	12-27-71	10
	23. Zion	Westinghouse Electric Corporation	NTR	50-87	R-119	01-28-72	10
Indiana	24. West Lafayette	Purdue U.	Lockheed	50-182	R-87	08-16-62	10

RESEARCH REACTORS – CONT.

State	Location	Licensee	Reactor Type	Docket	License Number	Date Issued	Authorized Power Level (KW)
Iowa	25. Ames	Iowa State U.	UTR-10	50-116	R-59	10-16-59	10
Kansas	26. Lawrence	U. of Kansas	Lockheed	50-148	R-78	06-23-61	250
	27. Manhattan	Kansas State U.	TRIGA	50-188	R-88	10-16-62	250
Maryland	28. College Park	U. of Maryland	TRIGA	50-166	R-70	10-14-60	250
	29. Bethesda	Armed Forces Radiobiology Research Institute	TRIGA	50-170	R-84	06-26-62	1000
Massachusetts	30. Lowell	U. of Lowell	GE	50-223	R-125	12-24-74	1000
	31. Cambridge	Massachusetts Institute of Technology	Heavy Water Reflected	50-20	R-37	06-09-58	5000
	32. Worcester	Worcester Polytechnic Institute	GE	50-134	R-61	12-16-59	10
Michigan	33. Ann Arbor	U. of Michigan	Pool	50-2	R-28	09-13-57	2000
	34. Lansing	Michigan State U.	TRIGA Mark I	50-294	R-114	03-21-69	250
	35. Midland	Dow Chemical Co.	TRIGA Mark I	50-264	R-108	07-03-67	100
Missouri	36. Rolla	U. of Missouri	Pool	50-123	R-79	11-21-61	200
	37. Columbia	U. of Missouri	Tank	50-186	R-103	10-11-66	10,000
Nebraska	38. Omaha	V.A. Hospital	TRIGA	50-131	R-57	06-26-59	18
New Mexico	39. Albuquerque	U. of New Mexico	AGN-201M No. 112	50-252	R-102	09-17-66	.005
New York	40. Buffalo	State U. of New York	PULSTAR	50-57	R-77	03-24-61	2000
	41. Sterling Forest	Union Carbide Corp.	Pool	50-54	R-81	09-07-61	5000
	42. New York City	Columbia U.	TRIGA Mark II	50-208	R-128	04-14-77	250
	43. Ithaca	Cornell U.	ZPR	50-97	R-89	12-11-62	.1
	44. Ithaca	Cornell U.	TRIGA Mk. II	50-157	R-80	01-11-62	100
	45. Riverdale	Manhattan College	Tank	50-199	R-94	03-24-64	.0001
North Carolina	46. Raleigh	North Caroline State U.	PULSTAR	50-297	R-120	08-25-72	1000
Ohio	47. Columbus	Ohio State U.	Pool	50-150	R-75	02-24-61	10
Oklahoma	48. Norman	U. of Oklahoma	AGN-211 No. 102	50-112	R-53	12-29-58	.015
Oregon	49. Corvallis	Oregon State U.	AGN-201 No. 114	50-108	R-51	11-13-58	.0001
	50. Corvallis	Oregon State U.	TRIGA Mark II	50-243	R-106	03-07-67	1000
	51. Portland	Reed Institute	TRIGA Mk. I	50-288	R-112	07-02-68	250
Pennsylvania	52. University Park	Pennsylvania State U.	TRIGA Mk. III	50-5	R-2	07-08-55	1000

best copy available

NON–POWER REACTORS IN THE U.S.

RESEARCH REACTORS – CONT.

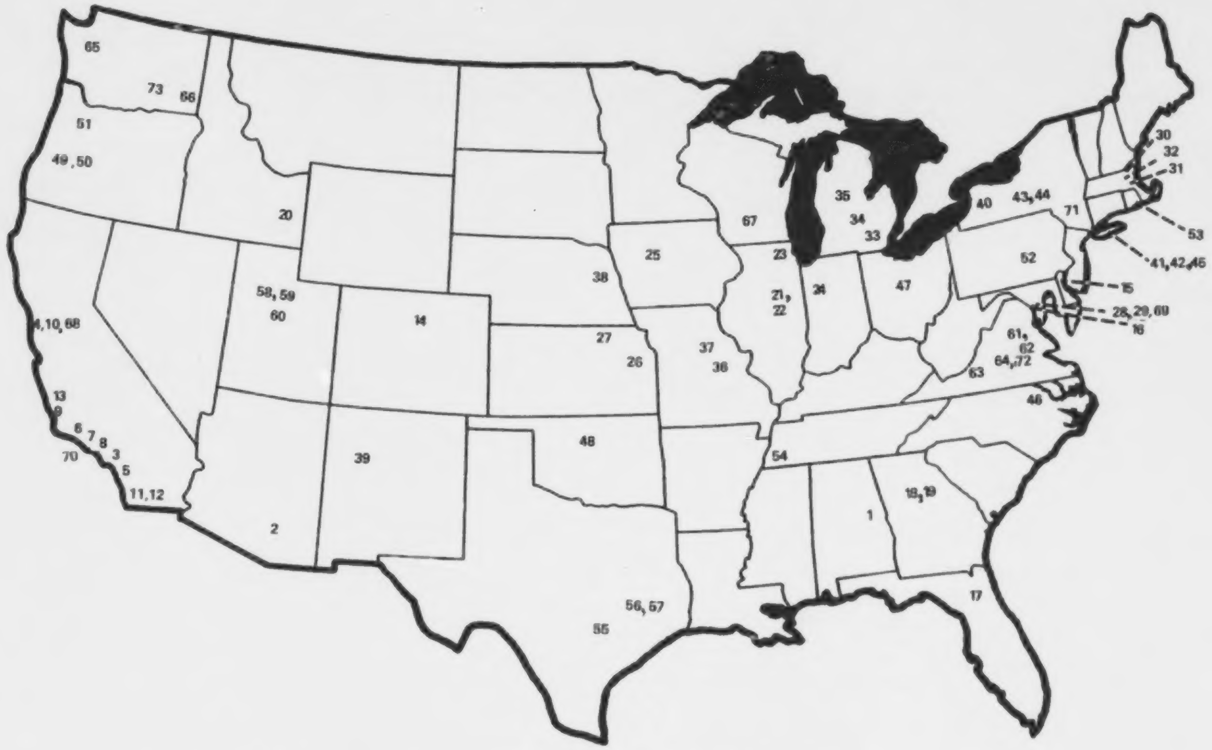
State	Location	Licensee	Reactor Type	Docket	License Number	Date Issued	Authorized Power Level (KW)
Rhode Island	53. Ft. Kearney, Narragansett	Rhode Island & Providence Plantations AEC	GE Pool	50-193	R-95	07-21-64	2000
Tennessee	54. Shelby County	Memphis State U.	AGN-201 No. 108	50-538	R-127	12-10-76	.0001
Texas	55. Austin	U. of Texas	TRIGA Mk. I	50-192	R-92	08-02-63	250
	56. College Station	Texas A&M U.	TRIGA	50-128	R-83	12-07-61	1000
	57. College Station	Texas A&M U.	AGN- 201M No. 106	50-59	R-23	08-26-57	.005
Utah	58. Salt Lake City	U. of Utah	AGN-201M No. 107	50-72	R-25	09-12-57	.005
	59. Salt Lake City	U. of Utah	TRIGA Mark I	50-407	R-126	09-30-75	100
	60. Provo	Brigham Young U.	L-77	50-262	R-109	09-07-67	.01
Virgina	61. Charlottesville	U. of Virgina	Pool	50-62	R-66	06-27-60	2000
	62. Charlottesville	U. of Virginia	Cavalier	50-396	R-123	09-24-74	.1
	63. Blacksburg	Virginia Polytechnic Institute	UTR-10	50-124	R-62	12-18-59	100
	64. Lynchburg	Babcock & Wilcox Co.	LPR	50-99	R-47	09-05-58	1000
Washington	65. Seattle	U. of Washington	Argonaut	50-139	R-73	03-31-61	100
	66. Pullman	Washington State U.	TRIGA	50-27	R-76	03-06-61	1000
Wisconsin	67. Madison	U. of Wisconsin	TRIGA	50-156	R-74	11-23-60	1000

2. EXPERIMENTAL AND TEST REACTORS

State	Location	Licensee	Reactor Type	Docket	License Number	Date Issued	Authorized Power Level (MWt)
California	68. Alameda Co.	General Electric Co.	GETR	50-70	TR-1	01-07-59	50
Maryland	69. Gaithersburg	National Bureau of Standards	TEST	50-184	TR-5	11-13-67 POL 06-30-70 OL	10

3. CRITICAL EXPERIMENT FACILITIES

State	Location	Licensee	Docket	License Number	Date Issued	Experiments
California	70. Ventura Co.	Rockwell International Corp. (Possesion Only)	50-147	CX-17	10-03-60	AETR Experiments (Terminal Requested).
New York	71. Schenectady	Rensselaer Polytechnic Institute	50-225	CX-22	07-03-64	
Virginia	72. Lynchburg	Babcock & Wilcox Co.	50-13	CX-10	10-22-58	NMSR Experiments
Washington	73. Richland	Battelle Memorial Institute	50-360	CX-26	11-29-71	Plutonium Recycle Critical Facility



Source: NRC Facilities License Application Record (FLAR) 06-30-77.

ABBREVIATIONS

AGN	Aerojet General Nuclear	These reactors were constructed by Aerojet at San Ramon, California, and then transferred to the current location.
ZPR	Zero Power Reactor	
LPR	Low Power Reactor	

DISTRIBUTION LIST–NRC

INTERNAL DISTRIBUTION

Office of the Commissioners	5
Atomic Safety and Licensing Appeal Panel	3
Advisory Committee on Reactor Safeguards	6
Office of Inspector and Auditor	1
Office of Policy Evaluation	1
Office of the General Counsel	1
Office of Public Affairs	21
Office of Congressional Affairs	3
Office of the Executive Director for Operations	7
Office of Administration	2
Office of the Executive Legal Director	2
Office of the Controller	2
Office for Analysis and Evaluation of Operational Data	2
Office of International Programs	3
Office of State Programs	1
Office of Management and Program Analysis	14
Office of Standards Development	1
Office of Nuclear Material Safety and Safeguards	2
– Division of Fuel Cycle and Material Safety	3
– Division of Safeguards	2
Office of Nuclear Reactor Regulation	5
– Division of Project Management	6
– Division of Operating Reactors	105
– Division of Systems Safety	4
– Division of Site Safety and Environmental Analysis	18
Office of Nuclear Regulatory Research	4
Office of Inspection and Enforcement	7
– Region I	11
– Region II	10
– Region III	10
– Region IV	10
– Region V	7
	279

EXTERNAL DISTRIBUTION

SPECIAL REQUESTS

Air Force	1
Congress	10
Department of Energy	32
Government Accounting Office	1
Environmental Protection Agency	1
Electric Power Research Institute	1
Argonne National Laboratory	3
Federal Power Commission	1
Bureau of Mines	1
Department of Agriculture	1
Department of Commerce	1
Department of the Interior	1
	54

OTHER

GPO Depository	440
GPO Stores	75
National Technical Information Service	25
Subscriptions (NTIS)	250
Colleges and Libraries (including Public Document Rooms)	128
Utilities and Other Requests	227
	1145

15/4-4

Nuclear Regulatory Commission

OPERATING UNITS STATUS REPORT

DATA AS OF 3-31-80

NUREG 0020
VOL. 4 NO. 4
APRIL 1980

NEWARK PUBLIC LIBRARY
JUN 18 1980
U. S. GOVT. PUBS.
DEPOSITORY 376

LICENSED OPERATING REACTORS

Y 3. N 88:

Available from
GPO Sales Program
Division of Technical Information and Document Control
U. S. Nuclear Regulatory Commission
Washington, D.C. 20555
and
National Technical Information Service
Springfield, Virginia 22161
703-326-8543
A Year's Subscription, 12 Issues, for this Publication is
$120.00 North American Continent ($240 00 Other).
Multiple Subscriptions Entered at the Same Time to the Same Address
Cost $90.00 North American Continent ($180.00 Other) for each Additional Subscription.

Single copies are $12.50 North American Continent ($25.00 Other).

This document is provided free of charge to those who participate in the data collection program, and to the Nuclear Regulatory Commission's local public document rooms.

Issued by the U.S. Nuclear Regulatory Commission

PUBLISHED: MAY 19, 1980

Nuclear Regulatory Commission

NUREG 0020
VOL. 4 NO. 4
APRIL 1980

OPERATING UNITS STATUS REPORT

DATA AS OF 3-31-80

LICENSED OPERATING REACTORS

AUTHORIZATION AND CLEARANCE

THE U.S. NUCLEAR REGULATORY COMMISSION'S OFFICE OF MANAGEMENT AND PROGRAM ANALYSIS PUBLISHES THIS MONTHLY STATUS REPORT "AS PART OF THE REPORTING REQUIREMENTS IN SECTION 50.36 OF 10 CFR PART 50 UNDER GAO CLEARANCE NUMBER B-180225, WITH AN EXPIRATION DATE OF SEPTEMBER 30, 1981," AS STATED IN THE OCTOBER 3, 1978 LETTER FROM JOHN M. LOVELADY, ASSISTANT DIRECTOR, GENERAL GOVERNMENT DIVISION, U. S. GENERAL ACCOUNTING OFFICE, TO J.M. FELTON, DIRECTOR, DIVISION OF RULES AND RECORDS, U.S. NUCLEAR REGULATORY COMMISSION.

STATEMENT OF PURPOSE

The U. S. Nuclear Regulatory Commission's monthly OPERATING UNITS STATUS REPORT – LICENSED OPERATING REACTORS, provides data on the operation of nuclear units as timely and accurately as possible. This information is collected by the Office of Management and Program Analysis from the Headquarters Staff of NRC's Office of Inspection and Enforcement, from NRC's Regional Offices, and from utilities. Since all of the data concerning operation of the units is provided by the utility operators less than two weeks after the end of the month, necessary corrections to published information are shown on the ERRATA page.

This report is divided into three sections: the first contains monthly highlights and statistics for commercial operating units, and errata from previously reported data; the second is a compilation of detailed information on each unit, provided by NRC Regional Offices and IE Headquarters; and the third section is an appendix containing nuclear availability and capacity factors, forced outage rate statistics, spent fuel storage capability, reactor-years of experience and non-power reactors in the United States.

Throughout the report, statistical factors for periods greater than a report month, or for more than one unit, are computed as the arithmetic average of each unit's individual operating statistics. The statistical factors for each unit for the report month are computed from actual power production for the month. The factors for the life-span of each unit (the "Cumulative" column) are reported by the utility and are not entirely re-computed by NRC. Utility power production data is checked for consistency with previously submitted statistics.

It is hoped this status report proves informative and helpful to all agencies and individuals interested in analyzing trends in the nuclear industry which might have safety implications, or in maintaining an awareness of the U. S. energy situation as a whole.

PHILIP A. ROSS
REVIEW COORDINATOR

TABLE OF CONTENTS

GLOSSARY ii
INDEX TO LICENSED UNITS iii

SECTION 1 – CURRENT DATA SUMMARIES

MONTHLY HIGHLIGHTS OF COMMERCIAL NUCLEAR POWER UNITS 1-2

- Licensed Power Reactors 1-2
- Power Generation 1-2
- Actual vs. Potential Energy Production 1-2
- Outage Data 1-2
- Derated Units 1-2
- Shutdowns - Number and Reasons 1-3
- Unit Availability, Capacity, and Forced Outage Rate 1-3
- Refueling Schedule 1-3

AVERAGE CAPACITY FACTORS BY VENDOR 1-4

AVERAGE DAILY POWER LEVEL FOR ALL COMMERCIAL OPERATING UNITS 1-5

ERRATA 1-7

SECTION 2 – OPERATING POWER REACTORS

ARKANSAS 1 THROUGH ZION 2 PAGES A–2 TO Z–5

- Facility Data
- Inspection and Enforcement Status
- Reports Received from Licensee
- Operating Status
- Average Daily Power Level (MWe)
- Unit Shutdowns/Reductions

SECTION 3 – APPENDIX

STATUS OF SPENT FUEL STORAGE CAPABILITY 3-2
REACTOR-YEARS OF EXPERIENCE 3-3
NON-POWER REACTORS IN THE U.S. 3-4

GLOSSARY

AVERAGE DAILY POWER LEVEL (MWe)	The net electrical energy generated during the day (measured from 0001 to 2400 hours inclusive) in megawatts hours divided by 24 hours.
LICENSED THERMAL POWER (MWt)	The maximum thermal power of the reactor authorized by the NRC, expressed in megawatts.
DATE OF COMMERCIAL OPERATION	Date unit was declared by utility owner to be available for the regular production of electricity; usually related to satisfactory completion of qualification tests as specified in the purchase contract and to accounting policies and practices of utility.
DESIGN ELECTRICAL RATING (DER) (Net MWe)	The nominal net electrical output of the unit specified by the utility and used for the purpose of plant design.
FORCED OUTAGE	An outage required to be initiated no later than the weekend following discovery of an off-normal condition.
FORCED OUTAGE HOURS	The clock hours during the report period that a unit is unavailable due to forced outages.
GROSS ELECTRICAL ENERGY GENERATED (MWH)	Electrical output of the unit during the report period as measured at the output terminals of the turbine generator, in megawatts hours.
GROSS HOURS	The clock hours from the beginning of a specified situation until its end. For outage durations, the clock hours during which the unit is not in power production.
GROSS THERMAL ENERGY GENERATED (MWH)	The thermal energy produced by the unit during the report period as measured or computed by the licensee in megawatt hours.
HOURS GENERATOR ON-LINE	Also, "Unit Service Hours." The total clock hours in the report period during which the unit operated with breakers closed to the station bus. These hours added to the total outage hours experienced by the unit during the report period, shall equal the hours in the report period.
HOURS IN REPORTING PERIOD	For units in power ascension at the end of the period, the gross hours from the beginning of the period or the first electrical production, whichever comes last, to the end of the period. For units in commercial operation at the end of the period, the gross hours from the beginning of the period or of commercial operation, whichever comes last, to the end of the period or decommissioning, whichever comes first.
HOURS REACTOR CRITICAL	The total clock hours in the report period during which the reactor sustained a controlled chain reaction.
MAXIMUM DEPENDABLE CAPACITY (GROSS) (MDC Gross) (Gross MWe)	Dependable main-unit gross capacity, winter or summer, whichever is smaller. The dependable capacity varies because the unit efficiency varies during the year due to cooling water temperature variations. It is the gross electrical output as measured at the output terminals of the turbine generator during the most restrictive seasonal conditions (usually summer).
MAXIMUM DEPENDABLE CAPACITY (NET) (MDC Net) (Net MWe)	Maximum Dependable Capacity (Gross) less the normal station service loads.
NAMEPLATE RATING (Gross MWe)	The nameplate power designation of the generator in megavolt amperes (MVA) times the nameplate power factor of the generator. NOTE: The nameplate rating of the generator may not be indicative of the maximum or dependable capacity, since some other item of equipment of a lesser rating (e.g., turbine) may limit unit output.
NET ELECTRICAL ENERGY GENERATED	Gross electrical output of the unit measured at the output terminals of the turbine generator during the reporting period, minus the normal station service electrical energy utilization. If this quantity is less than zero, a negative number should be recorded.
OUTAGE	A situation in which no electrical production takes place.
OUTAGE DATE	As reported on Appendix D of Reg. Guide 1.16, the date of the start of the outage. If continued from a previous month, report the same outage date but change "Method of Shutting Down Reactor" to "4 (Other)" and add a note: "Continued from previous month."
OUTAGE DURATION	The total clock hours of the outage measured from the beginning of the report period or the outage, whichever comes last, to the end of the report period or the outage, whichever comes first.
OUTAGE NUMBER	A number unique to the outage assigned by the licensee. The same number is reported each month in which the outage is in progress. One format is "76-05" for the fifth outage to occur in 1976.
PERIOD HOURS	See "Hours in Reporting Period."
POWER REDUCTION	A reduction in the Average Daily Power Level of more than 20% from the previous day. All power reductions are defined as outage of zero hours durations for the purpose of computing unit service and availability factors, and forced outage rate.

REACTOR AVAILABLE HOURS	The total clock hours in the report period during which the reactor was critical or was capable of being made critical. (Reactor Reserve Shutdown Hours + Hours Reactor Critical.)
REACTOR AVAILABILITY FACTOR	$\frac{\text{Reactor Available Hours} \times 100}{\text{Period Hours}}$
REACTOR RESERVE SHUTDOWN	The cessation of criticality in the reactor for administrative or other similar reasons when operation could have been continued.
REACTOR RESERVE SHUTDOWN HOURS	The total clock hours in the report period during which the reactor is in reserve shutdown mode. NOTE: No credit is given for NRC imposed shutdowns.
REACTOR SERVICE FACTOR	$\frac{\text{Hours Reactor Critical} \times 100}{\text{Period Hours}}$
REPORT PERIOD	Usually, the preceding calendar month. Can also be the preceding calendar year, (Year-to-Date), or the life-span of a unit (cumulative).
RESTRICTED POWER LEVEL	Maximum net electrical generation to which the unit is restricted during the report period due to the state of equipment, external conditions, administrative reasons, or a direction by NRC.
SCHEDULED OUTAGE	Planned removal of a unit from service for refueling, inspection, training, or maintenance. Those outages which do not fit the definition of "Forced Outage" perforce are "Scheduled Outages."
STARTUP AND POWER ASCENSION TEST PHASE	Period following initial criticality during which the unit is tested at successively higher levels, culminating with operation at full power for a sustained period and completion of warranty runs. Following this phase, the utility generally considers the unit to be available for commercial operation.
UNIT	The set of equipment uniquely associated with the reactor, including turbine generators, and ancillary equipment, considered as a single electrical energy production facility.
UNIT AVAILABLE HOURS	The total clock hours in the report period during which the unit operated on-line or was capable of such operation. (Unit Reserve Shutdown Hours + Hours Generator On-Line.)
UNIT AVAILABILITY FACTOR*	$\frac{\text{Unit Available Hours} \times 100}{\text{Period Hours}}$
UNIT CAPACITY FACTORS	
– Using Licensed Thermal Power	$\frac{\text{Gross Thermal Energy Generated} \times 100}{\text{Period Hours} \times \text{Lic. Thermal Power}}$
– Using Nameplate Rating	$\frac{\text{Gross Electrical Energy Generated} \times 100}{\text{Period Hours} \times \text{Nameplate Rating}}$
– Using DER	$\frac{\text{Net Electrical Energy Generated} \times 100}{\text{Period Hours} \times \text{DER}}$
– Using MDC Gross	$\frac{\text{Gross Electrical Energy Generated} \times 100}{\text{Period Hours} \times \text{MDC Gross}}$
– Using MDC Net	$\frac{\text{Net Electrical Energy Generated} \times 100}{\text{Period Hours} \times \text{MDC Net}}$

NOTE: if MDC Gross and/or MDC Net have not been determined. the DER is substituted for this quantity for Unit Capacity Factor calculations.

UNIT FORCED OUTAGE RATE	$\frac{\text{Forced Outage Hours}}{\text{Unit Service Hours} + \text{Forced Outage Hours}}$
UNIT RESERVE SHUTDOWN	The removal of the unit from on-line operation for economic or other similar reasons when operation could have been continued.
UNIT RESERVE SHUTDOWN HOURS	The total clock hours in the report period during which the unit was in reserve shutdown mode.
UNIT SERVICE FACTOR	$\frac{\text{Unit Service Hours} \times 100}{\text{Period Hours}}$
UNIT SERVICE HOURS	See "Hours Generator On-Line."

INDEX TO LICENSED UNITS

ARKANSAS 1 A-3
ARKANSAS 2 A-5
BEAVER VALLEY 1 B-3
BIG ROCK POINT 1 B-5
BROWNS FERRY 1 B-7
BROWNS FERRY 2 B-9
BROWNS FERRY 3 B-11
BRUNSWICK 1 B-13
BRUNSWICK 2 B-15
CALVERT CLIFFS 1 C-3
CALVERT CLIFFS 2 C-5
COOK 1 C-7
COOK 2 C-9
COOPER STATION C-11
CRYSTAL RIVER 3 C-13
DAVIS-BESSE 1 D-3
DRESDEN 1 D-5
DRESDEN 2 D-7
DRESDEN 3 D-9
DUANE ARNOLD D-11
FARLEY 1 F-3
FITZPATRICK F-5
FORT CALHOUN 1 F-7
FORT ST VRAIN F-9
GINNA G-3
HADDAM NECK H-3
HATCH 1 H-5
HATCH 2 H-7
HUMBOLDT BAY H-9
INDIAN POINT 1 I-3
INDIAN POINT 2 I-5
INDIAN POINT 3 I-7
KEWAUNEE K-3
LA CROSSE L-3
MAINE YANKEE M-3
MILLSTONE 1 M-5
MILLSTONE 2 M-7
MONTICELLO M-9
NINE MILE POINT 1 N-3
NORTH ANNA 1 N-5
OCONEE 1 O-3
OCONEE 2 O-5
OCONEE 3 O-7
OYSTER CREEK 1 O-9
PALISADES P-3
PEACH BOTTOM 2 P-5
PEACH BOTTOM 3 P-7
PILGRIM 1 P-9
POINT BEACH 1 P-11
POINT BEACH 2 P-13
PRAIRIE ISLAND 1 P-15
PRAIRIE ISLAND 2 P-17
QUAD CITIES 1 Q-3
QUAD CITIES 2 Q-5
RANCHO SECO 1 R-3
ROBINSON 2 R-5
SALEM 1 S-3
SAN ONOFRE 1 S-5
ST LUCIE 1 S-7
SURRY 1 S-9
SURRY 2 S-11
THREE MILE ISLAND 1 T-3
THREE MILE ISLAND 2 T-5
TROJAN T-7
TURKEY POINT 3 T-9
TURKEY POINT 4 T-11
VERMONT YANKEE 1 V-3
YANKEE-ROWE 1 Y-3
ZION 1 Z-3
ZION 2 Z-5

SECTION 1 — CURRENT DATA SUMMARIES

MONTHLY HIGHLIGHTS OF COMMERCIAL NUCLEAR POWER UNITS

FOR MONTH OF

MARCH 1980

LICENSED POWER REACTORS:

NUMBER		CAPACITY [1] MWe (Net)
67	IN COMMERCIAL OPERATION	49,023
67 [2]	LICENSED TO OPERATE	49,023
1 [3]	LICENSED FOR FUEL LOADING AND LOW POWER TESTING	

POWER GENERATION	REPORT MONTH	PREVIOUS MONTH	YEAR-TO-DATE
1. GROSS ELECTRICAL (MWHE)	20,881,056	20,287,108	61,207,318
2. NET ELECTRICAL (MWHE)	19,914,705	19,331,056	58,332,493
3. AVG. UNIT SERVICE FACTOR (%)	59.3	63.1	61.3
4. AVERAGE UNIT AVAILABILITY FACTOR (%)	59.6	63.1	61.6
5. AVERAGE UNIT CAPACITY FACTOR (MDC) (%)	53.9	56.4	56.0
6. AVG. UNIT CAPACITY FACTOR (DER) (%)	52.6	54.9	53.6
7. FORCED OUTAGE RATE (%)	11.6	10.4	10.5

NOTES

[1] BASED UPON MAXIMUM DEPENDABLE CAPACITY; DESIGN ELECTRIC RATING USED IF MDC NOT DETERMINED.

[2] EXCLUDES INDIAN POINT 1 (DER = 265), HUMBOLDT BAY (DER = 65), AND THREE MILE ISLAND 2 (DER = 906) WHICH HAVE OPERATING LICENSES BUT ARE SHUTDOWN INDEFINITELY.

[3] SEQUOYAH 1 (DER =1148) WAS LICENSED ON FEBRUARY 29, 1980 FOR FUEL LOADING AND LOW POWER TESTING ONLY.

ACTUAL VS. POTENTIAL ENERGY PRODUCTION

			% OF POTENTIAL PRODUCTION
POTENTIAL ENERGY PRODUCTION IN THIS PERIOD BY UNITS IN COMMERCIAL OPERATION (Using Maximum Dependable Capacity Net)		35,925,912 MWHe	
ENERGY ACTUALLY PRODUCED DURING THIS REPORT PERIOD	=	19,914,705 MWHe NET	55.4
ENERGY NOT PRODUCED DURING THIS REPORT PERIOD DUE TO SCHEDULED OUTAGES (NET)	=	11,334,698 MWHe	31.6
ENERGY NOT PRODUCED DURING THIS REPORT PERIOD DUE TO FORCED OUTAGES (NET)	=	3,046,737 MWHe	8.5
ENERGY NOT PRODUCED DURING THIS REPORT PERIOD FOR OTHER REASONS (NET)	=	1,629,772 MWHe	4.5
TOTAL		35,925,912 MWHe =	100%
ENERGY NOT PRODUCED THIS REPORT PERIOD DUE TO NRC IMPOSED SHUTDOWNS AND RESTRICTIONS	=	2,967,216 MWHe	8.3%
NUMBER OF UNITS WITH POWER RESTRICTIONS IMPOSED BY NRC	=	1	

OUTAGE DATA

	NUMBER	HOURS	PERCENT OF CLOCK TIME	MWHE LOST PRODUCTION
1. FORCED OUTAGES DURING MONTH	52	4,565.7	9.3	3,046,737
2. SCHEDULED OUTAGES DURING THE MONTH	37	16,074.1	32.6	11,334,698
3. TOTAL	89	20,639.8	41.9	14,381,435

MWHE LOST PRODUCTION = Down time x maximum dependable capacity net

DERATED UNITS

UNIT NAME	MAXIMUM DEPENDABLE CAPACITY (MWe - NET)	POWER LIMIT (MWe - NET)
BIG ROCK POINT 1	63	63
CALVERT CLIFFS 1	810	790
†FORT ST. VRAIN	330	231
INDIAN POINT 2	864	834
INDIAN POINT 3	965	917
POINT BEACH 1	495	390

†Restricted by NRC.

MONTHLY HIGHLIGHTS OF COMMERCIAL NUCLEAR POWER UNITS

FOR MONTH OF

MARCH 1980

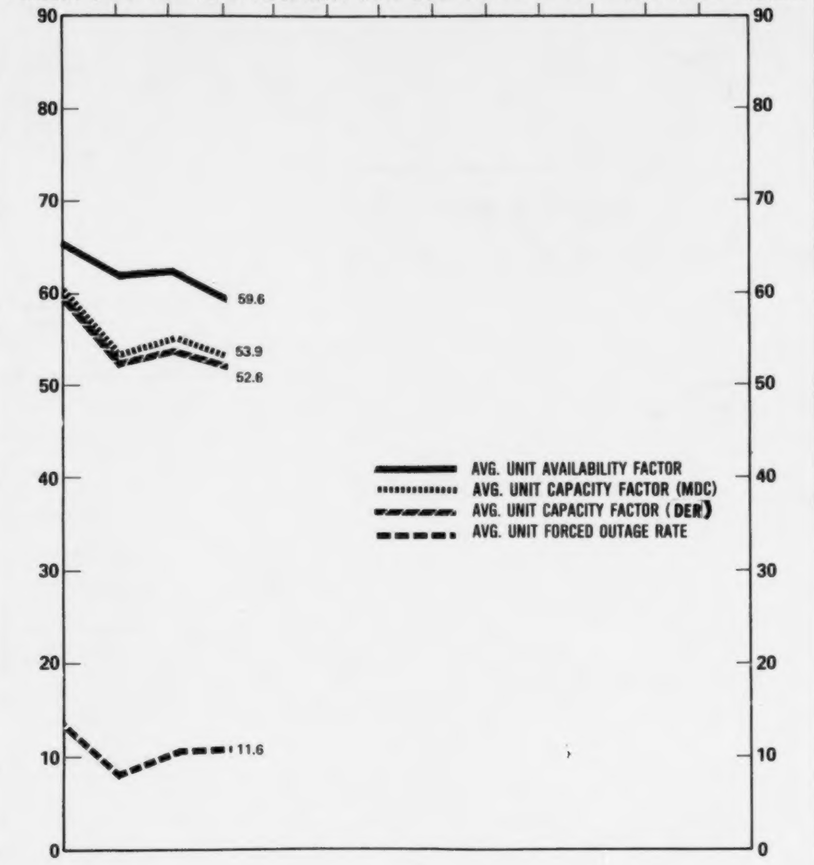
REASONS FOR SHUTDOWN

REASON	NUMBER	TOTAL HOURS LOST
A - Equipment Failure	35	3,237.0
B - Maintenance or Test	14	2,010.9
C - Refueling	17	10,187.0
D - NRC Restriction	7	4,159.2
E - Operator Training and License Examination	1	1.2
F - Administrative	5	28.8
G - Operational Error	2	26.1
H - Other	8	989.6
TOTAL	89	20,639.8

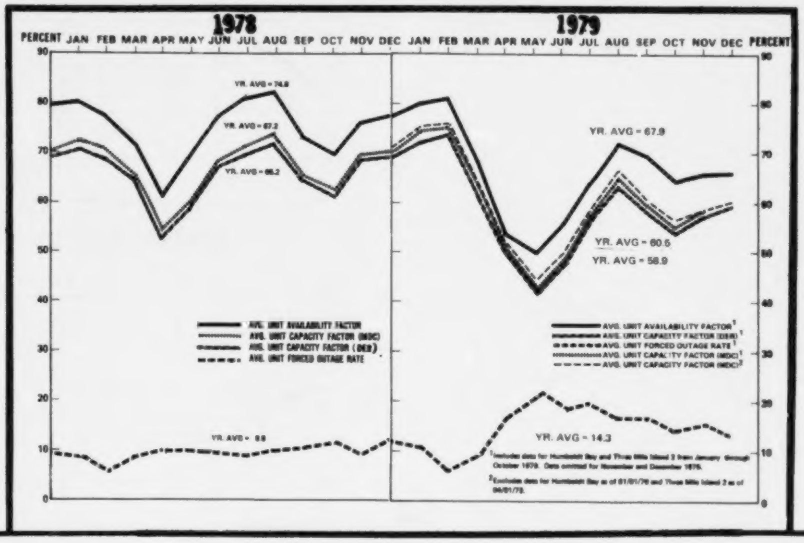
SHUTDOWNS GREATER THAN 72 HOURS EACH

UNIT NAME	REASON	UNIT NAME	REASON
ARKANSAS 1	A	NINE MILE POINT 1	H
ARKANSAS 2	B	OCONEE 2	C
BEAVER VALLEY 1	D	OCONEE 3	D
BROWNS FERRY 1	A, C	OYSTER CREEK 1	C
BRUNSWICK 2	C	PALISADES	D
COOPER STATION	C	PEACH BOTTOM 2	B, C
CRYSTAL RIVER 3	A, C	PEACH BOTTOM 3	2A
DAVIS-BESSE 1	A	PILGRIM 1	C
DRESDEN 1	H	POINT BEACH 1	D
DRESDEN 3	C	POINT BEACH 2	A
DUANE ARNOLD	C	QUAD CITIES 2	C
FORT CALHOUN 1	C	RANCHO SECO 1	C
FORT ST VRAIN	A, B	ROBINSON 2	B
HATCH 2	B	ST LUCIE 1	C
INDIAN POINT 3	B	SURRY 1	D
MAINE YANKEE	C	SURRY 2	A
MONTICELLO	C	THREE MILE ISLAND	D
		YANKEE-ROWE 1	A

UNIT AVAILABILITY, CAPACITY AND FORCED OUTAGE



UNIT AVAILABILITY, CAPACITY AND FORCED OUTAGE

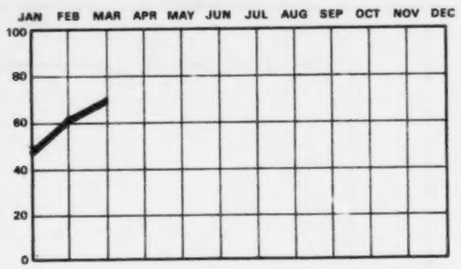


REFUELING SCHEDULE

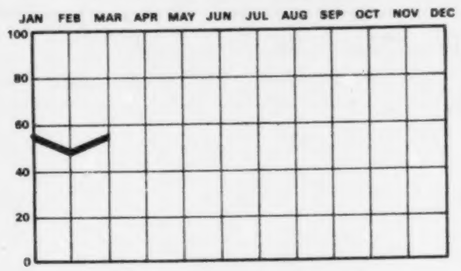
Please refer to the table on page 3–6 for the scheduled refueling date for each nuclear powerplant in this publication.

VENDOR CAPACITY FACTORS

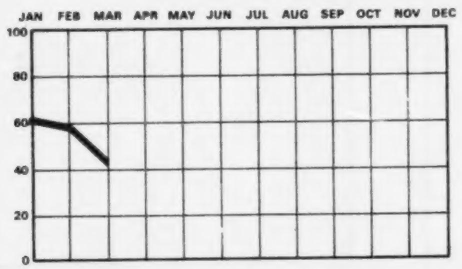
WESTINGHOUSE



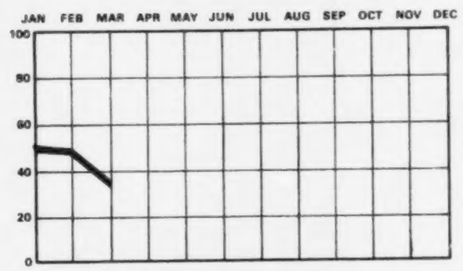
COMBUSTION ENGINEERING



GENERAL ELECTRIC



BABCOCK & WILCOX



MARCH 1980

OPERATIONS

1. COMPUTED BY ARITHMETIC AVERAGE OF UNIT CAPACITY FACTORS

GE BWR's	CFMDC
BROWNS FERRY 1	0.3
BROWNS FERRY 2	72.8
BROWNS FERRY 3	99.2
BRUNSWICK 1	73.2
BRUNSWICK 2	.0
COOPER STATION	2.0
DRESDEN 1	.0
DRESDEN 2	94.0
DRESDEN 3	.0
DUANE ARNOLD	.0
FITZPATRICK	87.5
HATCH 1	99.1
HATCH 2	.0
MILLSTONE 1	99.7
MONTICELLO	.0
NINE MILE POINT 1	67.4
OYSTER CREEK 1	.0
PEACH BOTTOM 2	41.9
PEACH BOTTOM 3	62.8
PILGRIM 1	.0
QUAD CITIES 1	92.6
QUAD CITIES 2	.0
VERMONT YANKEE 1	100.5

23 UNITS

W PWR's	CFMDC
BEAVER VALLEY 1	.0
COOK 1	99.8
COOK 2	96.1
FARLEY 1	90.1
GINNA	81.9
HADDAM NECK	100.6
INDIAN POINT 2	84.3
INDIAN POINT 3	36.1
KEWAUNEE	99.5
NORTH ANNA 1	98.0
POINT BEACH 1	.0
POINT BEACH 2	52.1
PRAIRIE ISLAND 1	95.1
PRAIRIE ISLAND 2	101.6
ROBINSON 2	54.0
SALEM 1	98.7
SAN ONOFRE 1	97.0
SURRY 1	.0
SURRY 2	.0
TROJAN	63.3
TURKEY POINT 3	98.6
TURKEY POINT 4	96.5
YANKEE-ROWE 1	.0
ZION 1	89.0
ZION 2	95.9

25 UNITS

CE PWR's	CFMDC
ARKANSAS 2	70.8
CALVERT CLIFFS 1	91.1
CALVERT CLIFFS 2	100.4
FORT CALHOUN 1	0.0
MAINE YANKEE	28.5
MILLSTONE 2	84.7
PALISADES	0.0
ST LUCIE 1	46.5

8 UNITS

B&W PWR's	CFMDC
ARKANSAS 1	77.5
CRYSTAL RIVER 3	.0
DAVIS-BESSE 1	62.9
OCONEE 1	96.7
OCONEE 2	6.8
OCONEE 3	43.0
RANCHO SECO 1	.0
THREE MILE ISLAND 1	.0

8 UNITS

2. COMPUTED BY RATIO OF VENDOR TOTALS

	GE BWR's	W PWR's	CE PWR's	B&W PWR's	All PWR's
NET ELECTRICAL PRODUCTION	5,921,009	9,797,409	2,243,943	1,835,380	13,876,732
MDC NET	17,327	18,428	6,090	6,737	31,255
CFMDC	45.9	71.5	56.3	36.6	61.1

Capacity factor in this page, denoted as CFMDC, is a function of the net maximum dependable capacity. See the corresponding definition in the Glossary. The vendor averages are computed by the formula :

$$\frac{\text{Net Electrical Energy Produced by Vendor}}{\text{Potential Electrical Production By Vendor In This Month.}} \quad X\ 100\%$$

Units Excluded	Reason for Exclusion
Indian Point 1 Humboldt Bay Three Mile Island 2	Unit Shutdown Indefinitely
Big Rock Point	Not Representative of Typical BWRs
LaCrosse	Not Representative of Typical BWRs and Atypical Vendor
Fort St. Vrain	Atypical Vendor and Reactor Design

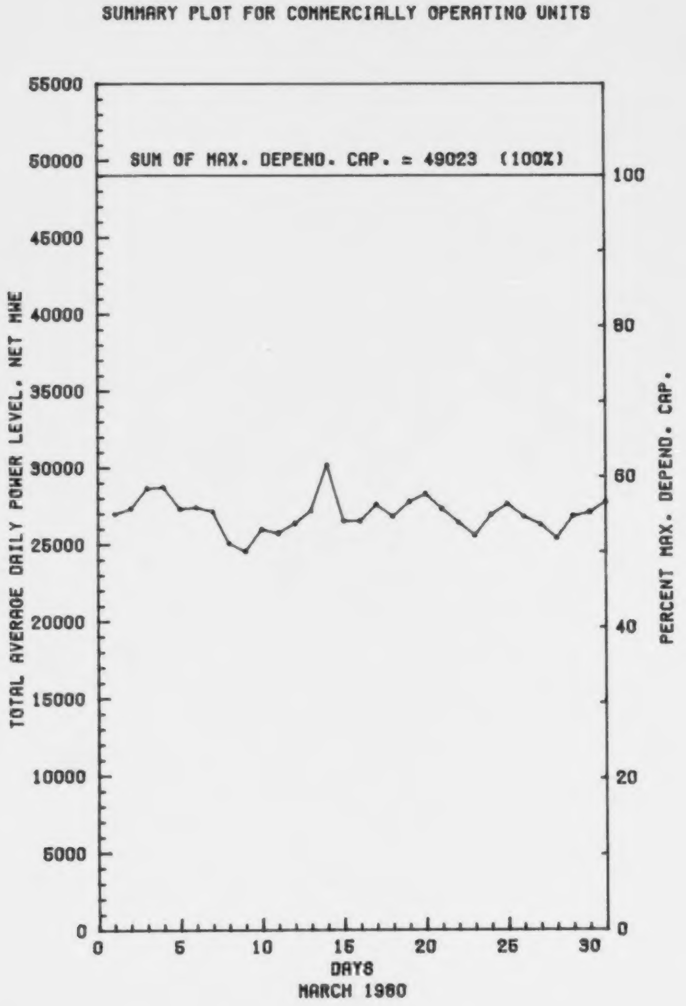
AVERAGE DAILY POWER LEVEL FOR ALL COMMERCIALLY OPERATING UNITS

This chart depicts the average daily power level for the units in commercial operation during the month.

The straight line on the graph labelled "SUM OF MAX DEP CAP'S" is plotted at the value shown by summing the separate maximum dependable capacities of the commercially operating units (in Net MWe). The plot shown below the line is calculated by summing the separate average daily power levels of the commercially operating units for each day of the month.

The scale on the left vertical axis runs in 1,000 MWe increments from 0 to 55,000 MWe (Net). The right vertical axis shows the percentage in 10% increments, up to 100% of the "SUM OF MAX DEP CAP'S."

It should be recognized that the 100% line would be obtainable only if all of the commercially operating units operated at 100% capacity, 24 hours per day, for the entire month. In other words, since any power generator must occasionally shut down to refuel and/or perform needed maintenance, and also since 100% capacity production is not always required by power demands, the 100% line is a theoretical goal and not a practical one.



ERRATA

Corrections and Amendments to Previously-Reported Data

(NOTE: These changes are reflected in the data contained in the current report)

CORRECTIONS TO NUREG C[illegible] VOL. 4 NO. 3 MARCH 1980

Page	Unit Name	Item	Correction
C-5	Calvert Cliffs 2	16. Gross Thermal Energy (MWH)	The February entry should have been 1,772,467 not the published value of 3,772,467. Year and cumulative values should be decreased by 2,000,000.
C-9	Cook 2	16. Gross Thermal Energy (MWH)	The correct value for the month of February is 2,165,634.
H-5	Hatch I	16. Gross Thermal Energy (MWH)	The correct value for the month of February should be 1,266,310 not 1,266,120.
H-7	Hatch 2	16. Gross Thermal Energy (MWH)	The correct value should be 1,588,770 rather than the published value of 1,588,320.
N-5	North Anna 1	16. Gross Thermal Energy (MWH)	The February entry should have been 1,483,418 rather than 1,690,114.

MULTIPLE - ISSUE CORRECTIONS

Fort St. Vrain	Item 12. Hours reactor was critical should be changed from 36.0 to 36.3 for the month of January 1980.
Maine Yankee	The licensee has recomputed the lifetime value of gross thermal energy.
North Anna	The monthly entries for Item 13. Reactor reserve shutdown hours for the lifetime of the unit should be:

6/78	2.9 Hours
7/78	19.5
8/78	0
9/78	1.0
10/78	54.1
11/78	3.2
12/78	16.1
1/79	0
2/79	28.4
3/79	8.5
4/79	0
5/79	11.8
6/79	0
7/79	0
8/79	0
9/79	2.8
10/79	0
11/79	0
12/79	0

SECTION 2 – OPERATING POWER REACTORS

ARKANSAS 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Arkansas/Pope 6 miles NW of Russellville
2.	Electric Reliability Council	Southwest Power Pool
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2568
5.	Design Electrical Rating (MWE Net)	850
6.	Date of Initial Criticality	August 6, 1974
7.	Date of Commercial Operation	December 19, 1974
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Dardanelle Reservoir

Utility & Contractor Information

10.	Licensee	Arkansas Power and Light Company
11.	Corporate Address	P.O. Box 551 Little Rock, Arkansas 72203
12.	Corporate Contact	W. Cavanaugh III Vice Pres., Generation & Construction
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	B&W
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	IV
17.	IE Branch Chief	G. L. Madsen
18.	IE Senior Inspector	T. F. Westerman
19.	IE Resident Inspector	W. D. Johnson (501) 968-3290
20.	Licensing Project Manager	G. S. Vissing
21.	Docket Number	50-313
22.	License Number/Date of Issue	DPR-51, 5/21/74
23.	Public Document Room	Arkansas Polytechnic College Russellville, Arkansas 72801

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO 10 CFR PART 20.203(C), "HIGH RADIATIONAREAS" DOOR 194 WAS FOUND TO BE UNLOCKED AND OPEN WITH NO POSITIVE ACCESS CONTROL ON FEBRUARY 6, 1980. DOOR 194 PROVIDES ACCESS TO THE UNIT 1 NORTH PIPING PENETRATION ROOM WHICH WAS A POSTED HIGH RADIATION AREA WITH ACCESSIBLE WHOLE-BODY RADIATION IN EXCESS OF 700 MILLIREMS PER HOUR. (8003 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NUMBER 2 LOW PRESSURE TURBINE HAS DISKS REMOVED DUE TO STRESS CRACKING DISCOVERED DURING THE LAST OUTAGE. GENERATOR OUTPUT IS REDUCED BY ABOUT 10%.

FACILITY ITEMS (PLANS AND PROCEDURES):

MANAGERIAL ITEMS

PLANT STATUS:

+ POWER OPERATIONS.

LAST IE SITE INSPECTION DATE: JANUARY 31 AND FEBRUARY 1, 1980

INSPECTION REPORT NO: 50-313/80-04 +

SUMMARY

INSPECTION SUMMARY

+ INVESTIGATION ON JANUARY 31 AND FEBRUARY 1, 1980 (80-04): SPECIAL, UNANNOUNCED INVESTIGATION OF ALLEGATIONS REGARDING PRACTICES OF THE LICENSEE'S HEALTH PHYSICS PERSONNEL AND MAINTENANCE CONTRACTED PERSONNEL, DURING THE PAST YEAR, THAT ARE CONSIDERED TO BE RADIOLOGICAL HAZARDS. NO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

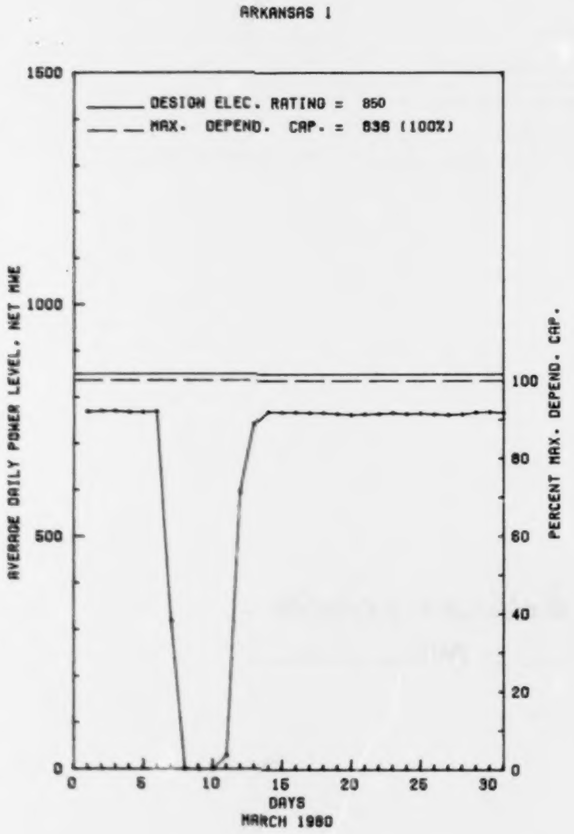
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-04 03-L0	2/5/80	3/6/80	REACTOR BUILDING ISOLATION VALVES EXCESSIVE LEAK RATE

REVIEWED BY OMPA LOEB: P S Robe **DATE:** 5/12/80

REVIEWED BY NRR DOR: **DATE:** 5/13/80

ARKANSAS 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-313
2. Utility Contact: L. S. Bramlett (501) 968-2519
3. Licensed Thermal Power (MWt): 2568
4. Nameplate Rating (Gross MWe): 1003 x 0.9 = 903
5. Design Electrical Rating (Net MWe): 850
6. Maximum Dependable Capacity (Gross MWe): 883
7. Maximum Dependable Capacity (Net MWe): 836
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	2,184.0	46,315.0
2. Hours Reactor Was Critical	650.8	1,144.1	31,282.3
13. Reactor Reserve Shutdown Hours	93.2	93.2	4,340.6
14. Hours Generator On-Line	639.3	1,128.1	30,598.6
15. Unit Reserve Shutdown Hours	.0	.0	796.7
16. Gross Thermal Energy (MWH)	1,610,838	2,816,149	74,078,193
17. Gross Electrical Energy (MWH)	504,590	884,160	24,605,316
18. Net Electrical Energy (MWH)	482,014	844,356	23,479,392
19. Unit Service Factor	85.9	51.7	66.1
20. Unit Availability Factor	85.9	51.7	67.8
*21. Unit Capacity Factor (MDC Net)	77.5	46.2	60.6
22. Unit Capacity Factor (DER Net)	76.2	45.5	59.6
23. Unit Forced Outage Rate	14.1	8.5	15.4

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-01	03/07/80	F	104.7	A	1		HC	XXXXXX	THE CAUSE OF THE OUTAGE WAS A VACUUM LEAK IN THE MAIN CONDENSER DUE TO A FAILURE OF THE RUBBER EXPANSION JOINT CONNECTING THE LP-2 TURBINE TO THE CONDENSER. A REPAIR WAS MADE BY VULCANIZING THE EXPANSION JOINT.

SUMMARY

OPERATION WAS ROUTINE DURING THE REPORT PERIOD, EXCEPT FOR AN OUTAGE FROM 03/07/80 TO 03/11/80 TO REPAIR A MAIN CONDENSER VACUUM LEAK CAUSED BY A TEAR IN A RUBBER EXPANSION JOINT.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

ARKANSAS 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1. Location (state/county) (nearest population center)	Arkansas/Pope 6 miles NW of Russellville
2. Electric Reliability Council	Southwest Power Pool
3. Type of Reactor	PWR
4. Power Level (MWT)	2815
5. Design Electrical Rating (MWE Net)	912
6. Date of Initial Criticality	12-5-78
7. Date of Commercial Operation	3/26/80
8. Condenser Cooling Method	Recirculating
9. Condenser Cooling Water Source	Cooling Tower

Utility & Contractor Information

10. Licensee	Arkansas Power & Light Company
11. Corporate Address	P.O. Box 551 Little Rock, Arkansas 72203
12. Corporate Contact	W. Cavanaugh III Vice Pres., Generation & Construction
13. Architect/Engineer	Bechtel
14. Nuclear Steam Supply System	Combustion Engineering, Inc.
15. Constructor	Bechtel

Regulatory Information

16. IE Region responsible	IV
17. IE Branch Chief	G. L. Madsen
18. IE Senior Inspector	T. F. Westerman
19. IE Resident Inspector	W. D. Johnson (501) 968-3290
20. Licensing Project Manager	R. E. Martin 492-8087
21. Docket Number	50-368
22. License Number/Date of Issue	NFP-6, 9/1/78
23. Public Document Room	Arkansas Polytechnic College Russellville, Arkansas 72801

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

FACILITY ITEMS (PLANS AND PROCEDURES):

MANAGERIAL ITEMS:

PLANT STATUS:

+ THE UNIT RETURNED TO POWER OPERATIONS AND WAS DECLARED COMMERCIAL.

LAST IE INSPECTION DATE: JANUARY 31 AND FEBRUARY 1, 1980 +

INSPECTION REPORT NO: 50-368/80-04 +

SUMMARY

INSPECTION SUMMARY

+ INVESTIGATION ON JANUARY 31 AND FEBRUARY 1, 1980 (80-04): SPECIAL, UNANNOUNCED INVESTIGATION OF ALLEGATIONS REGARDING PRACTICES OF THE LICENSEE'S HEALTH PHYSICS PERSONNEL AND MAINTENANCE CONTRACTED PERSONNEL DURING THE PAST YEAR, THAT ARE CONSIDERED TO BE RADIOLOGICAL HAZARDS. NO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

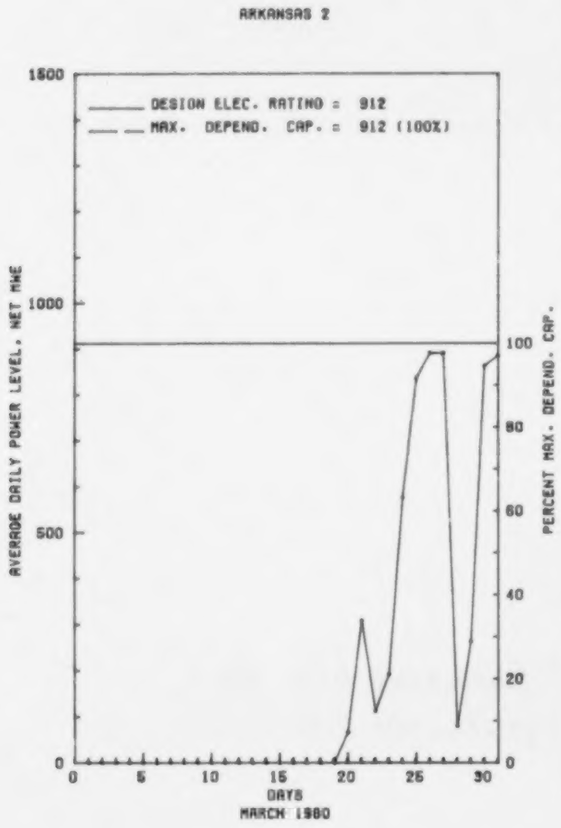
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-07 03-L0	1/24/80	2/22/80	SODIUM HYDROXIDE TANK LEVEL LOW
80-08 03-L0	1/28/80	2/22/80	CPC CHANNEL FAILURE
80-09 03-L0	1/29/80	2/28/80	REACTOR COOLANT SYSTEM ACTIVITY
80-10 03-L0	2/12/80	3/13/80	LATE PERFORMANCE OF LOCAL LEAK RATE TESTS
80-11 03-L0	2/14/80	3/13/80	CONTAINMENT PENETRATION EXCESSIVE LEAK RATES
80-12 03-L0	2/22/80	3/13/80	EMERGENCY DIESEL GENERATOR FAILURE

REVIEWED BY OMPA LOEB: AMR Beebe DATE: 05/12/80

REVIEWED BY NRR DOR: R E Martin DATE: 5/12/80

ARKANSAS 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-368
2. Utility Contact: Rex Pendergraft (501) 968-2519
3. Licensed Thermal Power (MWt): 2815
4. Nameplate Rating (Gross MWe): 959
5. Design Electrical Rating (Net MWe): 912
6. Maximum Dependable Capacity (Gross MWe): 912
7. Maximum Dependable Capacity (Net MWe): 912
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE *
11. Hours in Reporting Period	144.0	144.0	144.0
12. Hours Reactor Was Critical	124.0	124.0	124.0
13. Reactor Reserve Shutdown Hours	11.5	11.5	11.5
14. Hours Generator On-Line	112.6	112.6	112.6
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	296,229	296,229	296,229
17. Gross Electrical Energy (MWH)	97,510	97,510	97,510
18. Net Electrical Energy (MWH)	92,967	92,967	92,967
19. Unit Service Factor	78.1	78.1	78.1
20. Unit Availability Factor	78.1	78.1	78.1
21. Unit Capacity Factor (MDC Net)	70.8	70.8	70.8
22. Unit Capacity Factor (DER Net)	70.8	70.8	70.8
23. Unit Forced Outage Rate	22.0	22.0	22.0

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

*COMMERCIAL OPERATION STARTED ON 3/26/80. ALL OPERATING STATISTICS RESET TO ZERO AS OF THIS DATE.

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-04	01/29/80	S	456.6	B	4				SCHEDULED REACTOR TRIP PER POWER ESCALATION TESTING SEQUENCE
80-05	03/20/80	F	11.6	A	3		IA	INSTRU	"CPC" GENERATED TRIP. CEA 926 DROPPED. ROD WAS RECOVERED.
80-06	03/22/80	S	19.8	B	2				SCHEDULED TRIP PER T.P. 2.800.01 APPENDIX BB
03/26/80	DECLARATION OF COMMERCIAL OPERATION								
80-07	03/28/80	F	15.6	A	3		HC	INSTRU	MAIN FEEDWATER PUMP TRIP DUE TO LOSS OF LEVEL INDICATOR ON CONDENSER HOTWELL. INDICATOR WAS REPAIRED.
80-08	03/28/80	F	16.0	H	3		IA	INSTRU	"CPC" GENERATED RANGE LIMIT TRIP. - CPC TRIPPED REACTOR AT 0.6 ASI (AXIAL SHAPE INDEX).

SUMMARY

THE UNIT RETURNED TO POWER ON 03/20/80 FOLLOWING AN OUTAGE FOR TMI-RELATED MODIFICATIONS. UNIT TRIPS OCCURRED ON 03/20/80 DUE TO CONTROL ELEMENT ASSEMBLY CALCULATOR FAILURE; ON 03/22/80 FOR POWER ESCALATION TESTING; AND ON 03/28/80 FROM A CORE PROTECTION CALCULATOR RANGE LIMIT TRIP. A REACTOR TRIP ALSO OCCURRED ON 03/28/80 FOLLOWING A MAIN FEEDWATER PUMP TRIP FROM A LOOSE WIRE CONNECTION ON THE CONDENSER HOTWELL LEVEL INDICATOR.

LEGEND

[1]Type:
F--Forced
S--Scheduled

[2]Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3]Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4]Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5]Exhibit H:
Same source.

NRC POWER RESTRICTIONS: NONE

BEAVER VALLEY 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Pennsylvania/Beaver 5 miles E of E. Liverpool, Ohio
2.	Electric Reliability Council	East Central Area Reliability Coordination Agreement
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2652
5.	Design Electrical Rating (MWE Net)	852
6.	Date of Initial Criticality	May 10, 1976
7.	Date of Commercial Operation	10/01/76 (50%) 04/30/77 (100%)
8.	Condenser Cooling Method	Cooling tower/Closed loop
9.	Condenser Cooling Water Source	Ohio River

Utility & Contractor Information

10.	Licensee	Duquesne Light Company
11.	Corporate Address	435 Sixth Avenue Pittsburgh, Pennsylvania 15219
12.	Corporate Contact	C. N. Dunn, Vice President
13.	Architect/Engineer	Stone & Webster
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Senior Inspector	R. R. Keimig
19.	IE Resident Inspector	D. Beckman (412) 643-9200
20.	Licensing Project Manager	D. L. Wigginton
21.	Docket Number	50-334
22.	License Number/Date of Issue	DPR-66, 1/30/76
23.	Public Document Room	B. F. Jones Memorial Library 633 Franklin Avenue Aliquippa, PA 15001

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO TS 6.11.1.D, RECORDS OF PERFORMANCE OF OST 1.11.1, SI PUMP TEST (ISI-P-1A) FOR 9/10/79 HAVE NOT BEEN RETAINED.
(7927 DEFICIENCY)

CONTRARY TO TS 6.11 & BVPS RADCON MANUAL, APP 10, PART II.B.3, OUT-OF-SERVICE STICKER #3-362 FOR A RADCON RM-14 RATEMETER (FRISKER) WAS FOUND DISCARDED IN THE PRIMARY AUXILIARY BUILDING, THE ASSOCIATED INSTRUMENT HAVING BEEN RETURNED TO SERVICE W/O THE REQUIRED LOG ENTRIES HAVING BEEN COMPLETED.
(7927 DEFICIENCY)

DURING PERIOD 12/21/79 TO 1/7/80, PROCEDURE PT-10, REV. 1, WHICH WAS NEITHER REVIEWED BY OSC NOR APPROVED BY PLANT SUPERINTENDENT, WAS USED FOR PENETRANT EXAM. OF WELD 16 DM LOOP 2 COLD LEG NOZZLE TO SAFE-END FIELD WELD 24. TS 6.8.1.
(8002 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

FACILITY ITEMS (PLANS AND PROCEDURES):

Refueling/modification outage commenced on 11/30/79. The outage is expected to last until 7/80.

MANAGERIAL ITEMS:

A partial reorganization of station and corporate departments began on 12/16/79, and is substantially complete, pending further personnel assignments and approval of revised Tech. Specs.

PLANT STATUS:

Refueling completed on 3/1/80. In-service inspection, modifications and maintenance will continue through current outage.

LAST IE SITE INSPECTION DATE: 3/7/80

INSPECTION REPORT NO: 50-334/80-06

SUMMARY

INSPECTION SUMMARY

50-334/80-02 - Jan 8-11: Routine, unannounced inspection by one region-based inspector (29.5 hours). Inspection areas (3): inservice inspection (ISI) activities including ISI program review, review of implementing NDE procedures and ISI data review; activities associated with design change modifications; and licensee action on previous inspection findings. Noncompliances (1) (failure to review and approve NDE procedure prior to its implementation).

50-334/80-04 - Jan 22: Special public enforcement meeting convened by the Director, Office of Inspection and Enforcement, to discuss the licensee's actions pursuant to an Order Modifying License issued on December 5, 1979. The meeting was attended by senior licensee and NRC:OIE management and the public. A subsequent question and answer session was held to provide NRC responses to public concerns.

REPORTS RECEIVED FROM LICENSEE

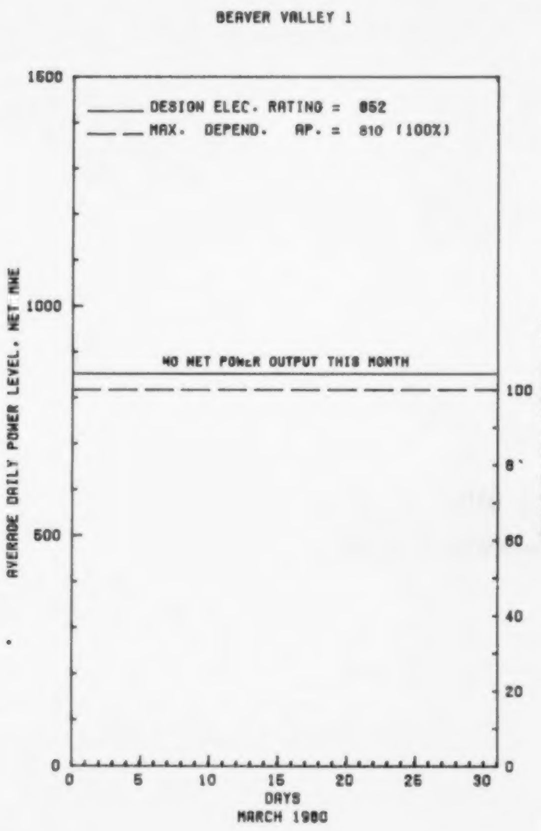
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-048/03L	12/10/79	02/07/80	30 Day LER	Dessicant bag in engine mounted fuel tank blocked suction line to diesel electric fuel pump
80-002/01T	01/17/80	01/30/80	14 Day LER	While in mode 5 with RCS level at mid-span in the loops suction was lost to all RHR pumps due to air binding
80-004/03L	01/23/80	02/07/80	30 Day LER	Radiation monitor setpoints less conservative than required for refueling but corrected prior to removing head from vessel
80-005/03L	01/12/80	02/07/80	30 Day LER	Operator entered restricted area without having rad tech or ratemeter with him
80-006/04T	01/30/80	02/07/80	14 Day LER	Tritium grab sample in Ohio River near plant discharge of 4240 picocuries/liter (1390 is reporting level) with no tritium releases above 10 CFR 20I limits

REVIEWED BY OMPA LOEB: T. C. Cintula DATE: 5·9·80

REVIEWED BY NRR DOR: DLWigginton DATE: 5·12·80

BEAVER VALLEY 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-334
2. Utility Contact: DAVID R. TIMKO (412) 643-5308
3. Licensed Thermal Power (MWt): 2652
4. Nameplate Rating (Gross MWe): 1026 x 0.9 = 923
5. Design Electrical Rating (Net MWe): 852
6. Maximum Dependable Capacity (Gross MWe): 845
7. Maximum Dependable Capacity (Net MWe): 810
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	2,184.0	34,344.0
12. Hours Reactor Was Critical	.0	.0	13,774.7
13. Reactor Reserve Shutdown Hours	.0	.0	4,482.7
14. Hours Generator On-Line	.0	.0	13,105.0
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	0	26,974,253
17. Gross Electrical Energy (MWH)	0	0	8,277,949
18. Net Electrical Energy (MWH)	0	0	7,570,743
19. Unit Service Factor	.0	.0	40.7
20. Unit Availability Factor	.0	.0	40.7
21. Unit Capacity Factor (MDC Net)	.0	.0	31.8
22. Unit Capacity Factor (DER Net)	.0	.0	30.3
23. Unit Forced Outage Rate	.0	.0	46.4

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
MAJOR MOD. & REFUELING (DECEMBER, 1979 THROUGH JULY 1980).
25. If Shut Down At End of Report Period, Estimated Date of Startup: 07/22/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
1	12/01/79	S	744.0	D	4		ZZ	ZZZZZZ	UNIT SHUTDOWN FOR MAJOR MODIFICATIONS AS REQUIRED BY THE NUCLEAR REGULATORY COMMISSION, INCLUDING NRC BULLETINS IEB 79-02 AND 79-14.

SUMMARY

THE UNIT REMAINED SHUTDOWN THROUGHOUT THE REPORT PERIOD FOR MODIFICATIONS AND PER NRC BULLETINS 79-02 AND 79-14.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

BIG ROCK POINT 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Michigan/Charlevoix 4 miles NE of Charlevoix
2.	Electric Reliability Council	East Central Area Reliability Coordination Agreement
3.	Type of Reactor	BWR
4.	Power Level (MWT)	240
5.	Design Electrical Rating (MWE Net)	72
6.	Date of Initial Criticality	September 27, 1962
7.	Date of Commercial Operation	March 29, 1963
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10.	Licensee	Consumers Power Company
11.	Corporate Address	212 West Michigan Avenue Jackson, Michigan 49201
12.	Corporate Contact	R. B. DeWitt
13.	Architect/Engineer	Bechtel Corporation
14.	Nuclear Steam Supply System	GE
15.	Constructor	Bechtel Corporation

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Senior Inspector	D.C. Boyd
19.	IE Principal Inspector	G. Wright
20.	Licensing Project Manager	W. A. Paulson
21.	Docket Number	50-155
22.	License Number/Date of Issue	DPR-6 issued 8/30/62
23.	Public Document Room	Charlevoix Public Library 107 Clinton Street Charlevoix, Michigan

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

Currently operating at 208 Mwt. The plant was shutdown during the first two weeks of January to make required short term lessons learned changes of NUREG 0578.

*LAST IE SITE INSPECTION DATE: February 6-8, 1980

* INSPECTION REPORT NO: 80-02

SUMMARY

INSPECTION SUMMARY

* Inspection on February 6-8, (80-02): Routine, unannounced inspection of plant operations, site organization and administration. The inspection involved 16 inspector hours, including four hours offshift, by one NRC inspector. Of the areas inspected, no items of noncompliance were identified.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

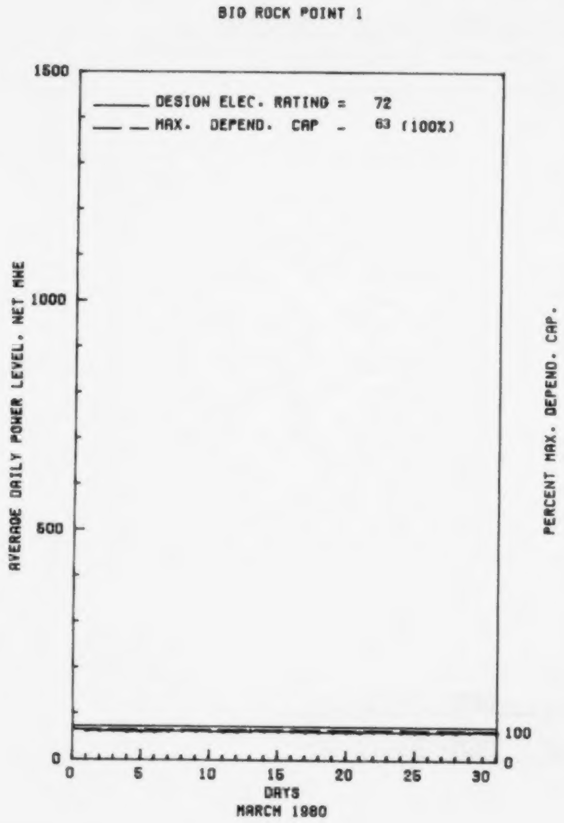
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
80-03/ 03L-0	01/23/80	02/21/80	30 Day LER	Following an automatic test circuit alarm indication, channel D of the RDS was removed from service for troubleshooting.
80-07/ 03L-0	02/24/80	03/19/80	30 Day LER	During routine operation at 1700 hours, excessive vibration was noted on control rod drive pump No. 2.

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 5-12-80

REVIEWED BY NRR DOR: Walter A Paulson **DATE:** 5-13-80

BIG ROCK POINT 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-155
2. Utility Contact: G. Himebauch (616) 547-6537 x180
3. Licensed Thermal Power (MWt): 240
4. Nameplate Rating (Gross MWe): 70.6 x 0.85 = 60
5. Design Electrical Rating (Net MWe): 72
6. Maximum Dependable Capacity (Gross MWe): 67
7. Maximum Dependable Capacity (Net MWe): 63
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): 63
10. Reasons for Restrictions, If Any:
FUEL THERMAL HYDRAULIC LIMITS

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	2,184.0	149,107.0
12. Hours Reactor Was Critical	744.0	1,862.2	101,995.2
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	744.0	1,849.5	99,848.4
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	159,107	389,455	18,507,386
17. Gross Electrical Energy (MWH)	48,374	118,689	5,854,607
18. Net Electrical Energy (MWH)	45,720	112,095	5,540,432
19. Unit Service Factor	100.0	84.7	67.0
20. Unit Availability Factor	100.0	84.7	67.0
21. Unit Capacity Factor (MDC Net)	97.5	81.5	59.0
22. Unit Capacity Factor (DER Net)	85.3	71.3	51.6
23. Unit Forced Cutage Rate	.0	2.1	24.9

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	NONE								

SUMMARY

OPERATION WAS ROUTINE THROUGHOUT THE REPORT PERIOD AT A RESTRICTION TO 63 MWE (NET) DUE TO FUEL THERMAL HYDRAULIC LIMITS.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

BROWNS FERRY 1

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Alabama/Limestone 10 miles NW of Decatur
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	3293
5.	Design Electrical Rating (MWE Net)	1065
6.	Date of Initial Criticality	August 17, 1973
7.	Date of Commercial Operation	August 1, 1974
8.	Condenser Cooling Method	Once Through With Forced Draft Cooling Towers
9.	Condenser Cooling Water Source	Tennessee River

Utility & Contractor Information

10.	Licensee	Tennessee Valley Authority
11.	Corporate Address	500A Chestnut Street Tower II Chattanooga, Tennessee 37401
12.	Corporate Contact	H. G. Parris, Manager of Power.
13.	Architect/Engineer	TVA
14.	Nuclear Steam Supply System	GE
15.	Constructor	TVA

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. Lewis
18.	IE Senior Inspector	H. C. Dance
19.	IE Resident Inspector	R. Sullivan (205) 729-6196
20.	Licensing Project Manager	R. Clark
21.	Docket Number	50-259
22.	License Number/Date of Issue	DPR-33, 6/26/73
23.	Public Document Room	Athens Public Library South and Forrest Athens, Alabama 35611

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

+ REFUELING OPERATIONS COMPLETED. UNIT SHUTDOWN AT END OF MONTH.

+ LAST IE SITE INSPECTION DATE: MARCH 7 - APRIL 4, 1980

+ INSPECTION REPORT NO: 50-259/80-19

SUMMARY

INSPECTION SUMMARY

+ INSPECTION DECEMBER 10 - JANUARY 4 (79-48): THIS ROUTINE INSPECTION INVOLVED 21 RESIDENT INSPECTOR-HOURS IN THE AREAS OF PLANT OPERATIONS, PLANT TOURS, REPORTABLE OCCURRENCES, REFUELING, MAINTENANCE, SURVEILLANCE TESTING, PLANT PHYSICAL PROTECTION, AND RADIATION AREA CONTROLS. OF THE EIGHT AREAS INSPECTED NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN SEVEN AREAS. ONE ITEM OF APPARENT NONCOMPLIANCE WAS FOUND IN ONE AREA (DEFICIENCY - USE OF TEST PROCEDURE WHICH HAD BEEN SUPERCEEDED BY REVISION).

INSPECTION FEBRUARY 6-8 (80-05): THIS SPECIAL, ANNOUNCED INSPECTION INVOLVED 5 INSPECTOR-HOURS ON SITE IN THE AREA OF SEISMIC ANALYSIS FOR AS-BUILT SAFETY-RELATED PIPING SYSTEMS (IE BULLETIN 79-14). NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION FEBRUARY 19-24 (80-08): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 59 INSPECTOR-HOURS ON SITE IN THE AREAS OF WITNESSING THE INTEGRATED LEAK RATE TESTING ON UNIT 1. OF THE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

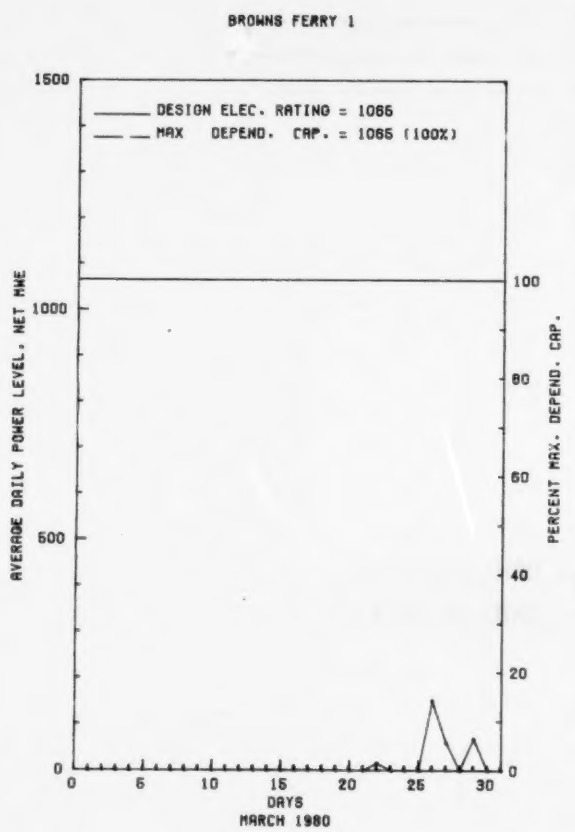
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
80-10/ 03L-0	01/29/80	02/27/80	30 DAY LER	HIGH PRESSURE COOLANT INJECTION SENSING LINE HANGERS BROKEN.
80-12/ 03L-0	02/01/80	02/29/80	30 DAY LER	ELECT AND MECH OVERSPEED TRIP MECHANISMS DISCOVERED DAMAGED DURING PREVENTATIVE MAINT ON RCIC TERRY TURBINE.
80-14/ 03L-0	01/26/80	02/25/80	30 DAY LER	SCRAM ACCUMULATOR LEVEL SWITCH INOPERABLE.
80-17/ 01T-0	02/21/80	03/05/80	24 DAY LER	PIPING SEISMIC RESTRAINT INADEQUATE.

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 5-12-80

REVIEWED BY NRR DOR: Dick Clark **DATE:** 05-13-80

BROWNS FERRY 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-259
2. Utility Contact: Don Green (205) 729-6846
3. Licensed Thermal Power (MWt): 3293
4. Nameplate Rating (Gross MWe): 1280 x 0.9 = 1152
5. Design Electrical Rating (Net MWe): 1065
6. Maximum Dependable Capacity (Gross MWe): 1098
7. Maximum Dependable Capacity (Net MWe): 1065
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	2,184.0	49,682.0
12. Hours Reactor Was Critical	106.2	177.9	28,455.2
13. Reactor Reserve Shutdown Hours	174.6	174.6	4,864.1
14. Hours Generator On-Line	41.5	113.2	27,726.1
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	22,356	206,506	74,342,999
17. Gross Electrical Energy (MWH)	8,030	69,990	24,588,450
18. Net Electrical Energy (MWH)	2,089	59,253	23,861,221
19. Unit Service Factor	5.6	5.2	55.8
20. Unit Availability Factor	5.6	5.2	55.8
21. Unit Capacity Factor (MDC Net)	.3	2.5	45.1
22. Unit Capacity Factor (DER Net)	.3	2.5	45.1
23. Unit Forced Outage Rate	81.5	61.7	33.2

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 4/1/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
108	01/03/80	S	519.8	C	4				REFUELING CONTINUED
109	03/22/80	F	76.4	A	2				BLOWN SEAL ON "B" RECIRC PUMP
110	03/27/80	F	54.8	A	3				GENERATOR NEUTRAL OVER VOLTAGE DUE TO PROBLEMS WITH MAIN TRANSFORMER 1B
111	03/29/80	F	51.5	A	2				HIGH TEMPERATURE ON "B" RECIRC PUMP THRUST BEARING.

SUMMARY

THE UNIT REMAINED SHUTDOWN UNTIL 03/22/80 FOR REFUELING AND EXPERIENCED PROBLEMS WITH THE MAIN TRANSFORMER 1B AND THE "B" RECIRCULATION PUMP THRUST BEARING FOLLOWING RETURN TO SERVICE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

BROWNS FERRY 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1. Location (state/county) (nearest population center)	Alabama/Limestone 10 miles NW of Decatur
2. Electric Reliability Council	Southeastern Electric Reliability Council
3. Type of Reactor	BWR
4. Power Level (MWT)	3293
5. Design Electrical Rating (MWE Net)	1065
6. Date of Initial Criticality	July 20, 1974
7. Date of Commercial Operation	March 1, 1975
8. Condenser Cooling Method	Once Through With Forced Draft Cooling Towers
9. Condenser Cooling Water Source	Tennessee River

Utility & Contractor Information

10. Licensee	Tennessee Valley Authority
11. Corporate Address	500A Chestnut Street Tower II Chattanooga, Tennessee 37401
12. Corporate Contact	H. G. Parris, Manager of Power
13. Architect/Engineer	TVA
14. Nuclear Steam Supply System	GE
15. Constructor	TVA

Regulatory Information

16. IE Region responsible	II
17. IE Branch Chief	R. C. Lewis
18. IE Senior Inspector	H. C. Dance
19. IE Resident Inspector	R. Sullivan (205) 729-6196
20. Licensing Project Manager	R. Clark
21. Docket Number	50-260
22. License Number/Date of Issue	DPR-52, 6/28/74
23. Public Document Room	Athens Public Library South and Forrest Athens, Alabama 35611

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

+ 100 PERCENT FULL POWER OPERATION.

+ LAST IE SITE INSPECTION DATE: MARCH 7 - APRIL 4, 1980

+ INSPECTION REPORT NO: 50-260/80-19

+ SUMMARY

INSPECTION SUMMARY

+ INSPECTION DECEMBER 10 - JANUARY 4 (79-47): THIS ROUTINE INSPECTION INVOLVED 21 RESIDENT INSPECTOR-HOURS IN THE AREAS OF PLANT OPERATIONS, PLANT TOURS, REPORTABLE OCCURRENCES, REFUELING, MAINTENANCE, SURVEILLANCE TESTING, PLANT PHYSICAL PROTECTION, AND RADIATION AREA CONTROLS. OF THE EIGHT AREAS INSPECTED NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN SEVEN AREAS. ONE ITEM OF APPARENT NONCOMPLIANCE WAS FOUND IN ONE AREA (DEFICIENCY - USE OF TEST PROCEDURE WHICH HAD BEEN SUPERCEEDED BY REVISION).

INSPECTION FEBRUARY 6-8 (80-05): THIS SPECIAL, ANNOUNCED INSPECTION INVOLVED 5 INSPECTOR-HOURS ON SITE IN THE AREA OF SEISMIC ANALYSIS FOR AS-BUILT SAFETY-RELATED PIPING SYSTEMS (IE BULLETIN 79-14). NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

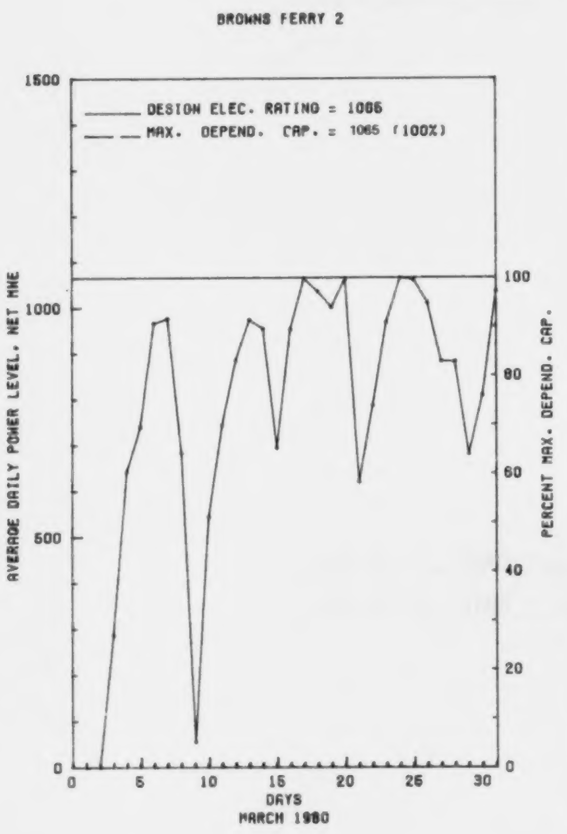
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
80-06/ 03L-0	02/03/80	03/03/80	30 DAY LER	2A RECIRC PUMP INADVERTENTLY TRIPPED WITH 2B OUT OF SERVICE.
80-07/ 03L-0	02/03/80	03/03/80	30 DAY LER	2B RECIRC PUMP TRIPPED WHEN MG SET STATOR TEMP. RELAY PICKED UP.
80-08/ 03L-0	02/06/80	03/06/80	30 DAY LER	RECIRC PUMP 2B DUE PERSONNEL ERROR DURING SURVEILLANCE. TEST.
80-09/ 03L-0	02/13/80	03/12/80	30 DAY LER	RAD MONITOR FAILED DUE TO SOLA TRANSFORMER FAILURE.

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 5-12-80

REVIEWED BY NRR DOR: Dick Clark **DATE:** 05-13-80

BROWNS FERRY 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-260
2. Utility Contact: Don Green (205) 729-6846
3. Licensed Thermal Power (MWt): 3293
4. Nameplate Rating (Gross MWe): 1280 x 0.9 = 1152
5. Design Electrical Rating (Net MWe): 1065
6. Maximum Dependable Capacity (Gross MWe): 1098
7. Maximum Dependable Capacity (Net MWe): 1065
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restrictud, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	2,184.0	44,593.9
12. Hours Reactor Has Critical	680.8	1,782.8	26,267.2
13. Reactor Reserve Shutdown Hours	63.2	401.1	12,176.0
14. Hours Generator On-Line	666.0	1,731.5	25,396.0
15. Unit Reserve Shutdown Hours	.0	.0	0
16. Gross Thermal Energy (MWH)	1,772,736	4,772,174	70,927,174
17. Gross Electrical Energy (MWH)	593,150	1,594,600	23,544,668
18. Net Electrical Energy (MWH)	577,104	1,549,941	22,881,410
19. Unit Service Factor	89.5	79.3	57.0
20. Unit Availability Factor	89.5	79.3	57.0
*21. Unit Capacity Factor (MDC Net)	72.8	66.6	48.2
Unit Capacity Factor (DER Net)	72.8	66.6	48.2
it Forced Outage Rate	10.5	20.7	35.9

-wns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

n At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
125	03/01/80	F	57.5	A	9				PRIMARY CONTAINMENT LEAKAGE. RX WAS PREVIOUSLY DOWN FOR HPCI MAINTENANCE.
126	03/07/80	S	.0	B	5				DERATED FOR RECIRC M.G. SET BRUSH REPLACEMENT.
127	03/09/80	F	20.5	B	3				RX LOW WATER LEVEL SIGNAL WHILE PERFORMING S.I. (APRM FUNCTIONAL)
128	03/14/80	S	.0	H	5				DERATED FOR CONTROL ROD SEQUENCE EXCHANGE.
129	03/21/80	F	.0	H	5				DERATED DUE TO RECIRC PUMP TRIP
130	03/27/80	F	.0	H	5				DERATED FOR MAINTENANCE TO HP HTRS AND RX FW PUMPS
131	03/29/80	F	.0	H	5				DERATED FOR HP FW HEATER MAINTENANCE.

SUMMARY

THE UNIT RETURNED TO SERVICE ON 03/03/80 FOLLOWING AN OUTAGE TO REPAIR THE HIGH PRESSURE COOLANT INJECTION TURBINE BEARING PEDESTALS. THE REACTOR SCRAMMED ON 03/09/80 WHEN "B" CHANNEL SOLENOIDS DE-ENERGIZED (CAUSE UNKNOWN), FOLLOWED BY A REACTOR LOW WATER ACTIVATED SCRAM.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

BROWNS FERRY 3

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Alabama/Limestone 10 miles NW of Decatur
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	3293
5.	Design Electrical Rating (MWE Net)	1065
6.	Date of Initial Criticality	August 8, 1976
7.	Date of Commercial Operation	March 1, 1977
8.	Condenser Cooling Method	Once Through With Forced Draft Cooling Tower
9.	Condenser Cooling Water Source	Tennessee River

Utility & Contractor Information

10.	Licensee	Tennessee Valley Authority
11.	Corporate Address	500A Chestnut Street Tower II Chattanooga, Tennessee 37401
12.	Corporate Contact	H. G. Parris, Manager of Power
13.	Architect/Engineer	TVA
14.	Nuclear Steam Supply System	GE
15.	Constructor	TVA

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R.C. Lewis
18.	IE Senior Inspector	H. C. Dance
19.	IE Resident Inspector	R. Sullivan (205) 729-6196
20.	Licensing Project Manager	R. Clark
21.	Docket Number	50-296
22.	License Number/Date of Issue	DPR-68, 7/2/76
23.	Public Document Room	Athens Public Library South and Forrest Athens, Alabama 35611

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

+ FULL POWER OPERATIONS.

+ LAST IE SITE INSPECTION DATE: MARCH 7 - APRIL 4, 1980

+ INSPECTION REPORT NO: 50-296/80-19

SUMMARY

INSPECTION SUMMARY

+ INSPECTION DECEMBER 10 - JANUARY 4 (79-47): THIS ROUTINE INSPECTION
INVOLVED 23 RESIDENT INSPECTOR-HOURS IN THE AREAS OF PLANT OPERATIONS,
PLANT TOURS, REPORTABLE OCCURRENCES, REFUELING, MAINTENANCE, SURVEIL-
LANCE TESTING, PLANT PHYSICAL PROTECTION, AND RADIATION AREA CONTROLS.
OF THE EIGHT AREAS INSPECTED NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS
WERE IDENTIFIED IN SEVEN AREAS. ONE ITEM OF APPARENT NONCOMPLIANCE
WAS FOUND IN ONE AREA (DEFICIENCY - USE OF TEST PROCEDURE WHICH HAD
BEEN SUPERCEEDED BY REVISION).

INSPECTION FEBRUARY 6-8 (80-05): THIS SPECIAL, ANNOUNCED INSPECTION INVOLVED 6 INSPECTOR-HOURS ON SITE IN THE AREA OF SEISMIC ANALYSIS FOR AS-BUILT SAFETY-RELATED PIPING SYSTEMS (IE BULLETIN 79-14). NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

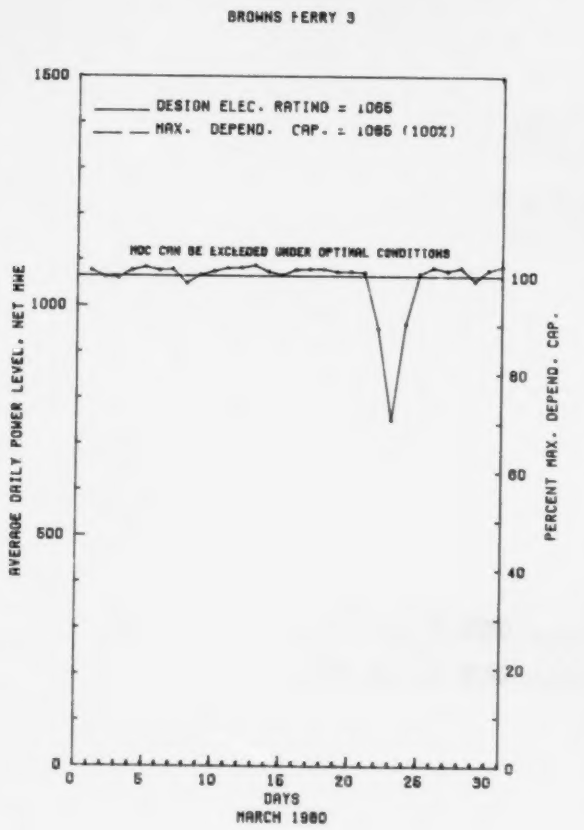
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
80-04/03L-0	01/29/80	02/27/80	30 DAY LER	4160 VOLT TIE BOARD DEENERGIZED.
80-05/03L	02/05/80	03/05/80	30 DAY LER	VALVE FCV 74-73 FOUND INOPERABLE DURING SURVEILLANCE TESTING.

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 5-12-80

REVIEWED BY NRR DOR: Dick Clark **DATE:** 05-13-80

BROWNS FERRY 3

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-296

2. Utility Contact: Don Green (205) 729-6846

3. Licensed Thermal Power (MWt): 3293

4. Nameplate Rating (Gross MWe): 1280 x 0.9 = 1152

5. Design Electrical Rating (Net MWe): 1065

6. Maximum Dependable Capacity (Gross MWe): 1098

7. Maximum Dependable Capacity (Net MWe): 1065

8. If Changes Occur in Capacity Ratings Above Since Last Report. Give Reasons:

NONE

9. Power Level To Which Restricted. If Any (Net MWe): NONE

10. Reasons for Restrictions. If Any:

NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	2,184.0	27,048.0
12. Hours Reactor Was Critical	744.0	1,772.4	20,680.1
13. Reactor Reserve Shutdown Hours	.0	.0	1,658.4
14. Hours Generator On-Line	744.0	1,748.9	20,186.7
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	2,375,681	5,487,881	58,595,747
17. Gross Electrical Energy (MWH)	804,870	1,854,790	19,257,370
18. Net Electrical Energy (MWH)	786,318	1,806,506	18,694,279
19. Unit Service Factor	100.0	80.1	74.6
20. Unit Availability Factor	100.0	80.1	74.6
21. Unit Capacity Factor (MDC Net)	99.2	77.7	64.9
22. Unit Capacity Factor (DER Net)	99.2	77.7	64.9
23. Unit Forced Outage Rate	.0	19.9	10.1

24. Shutdowns Scheduled Over Next 6 Months (Type.Date.and Duration of Each):

NONE

25. If Shut Down At End of Report Period. Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
118	03/08/80	S	.0	B	5				DERATED FOR TURBINE CONTROL VALVE TESTS AND SI'S.
119	03/22/80	S	.0	B	5				DERATED FOR RECIRC PUMPS M.G. SET BRUSH REPLACEMENT AND CONTROL ROD ADJUSTMENTS.

SUMMARY

OPERATION WAS AT NEAR FULL POWER DURING THE REPORT PERIOD, WITH A LOAD REDUCTION FROM 03/22/80 THROUGH 03/24/80 FOR RECIRCULATION PUMP MOTOR GENERATOR SET BRUSH REPLACEMENT AND CONTROL ROD ADJUSTMENT.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B Maintenance or Test
C Refueling
D Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

NRC POWER RESTRICTIONS: NONE

BRUNSWICK 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	North Carolina/Brunswick 3 miles N of Southport
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2436
5.	Design Electrical Rating (MWE Net)	821
6.	Date of Initial Criticality	October 8, 1976
7.	Date of Commercial Operation	March 18, 1977
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Cape Fear River

Utility & Contractor Information

10.	Licensee	Carolina Power and Light Company
11.	Corporate Address	411 Fayetteville Street Raleigh, North Carolina 27602
12.	Corporate Contact	J. A. Jones Executive Vice President
13.	Architect/Engineer	United Engineers & Constructors, Inc.
14.	Nuclear Steam Supply System	GE
15.	Constructor	Brown and Root, Inc.

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R.C. Lewis
18.	IE Senior Inspector	P.J. Kellogg
19.	IE Resident Inspector	J. E. Outzs (919) 457-9531
20.	Licensing Project Manager	J. Hannon
21.	Docket Number	50-325
22.	License Number/Date of Issue	DPR-71, 9/8/76
23.	Public Document Room	Southport-Brunswick County Library 109 W. Moore Street Southport, North Carolina 28461

INSPECTION STATUS

ENFORCEMENT STATUS

AS REQUIRED BY TECHNICAL SPECIFICATION 6.8.1, PROCEDURES SHALL BE WRITTEN AND IMPLEMENTED. PROCEDURTI-200, APPENDIX A, REQUIRES FORMAL CLASSROOM LECTURES FOR LICENSEES SCORING LESS THAN 80% IN A SPECIFIC AREA OF AN ANNUAL EXAMINATION. PROCEDURE TI-200, APPENDIX A, WAS NOT IMPLEMENTED IN THAT OF EIGHT (8) LECTURE SERIES CONDUCTED IN 1978, FOUR (4) WERE CONDUCTED LARGELY OR IN PART AS SELF-STUDY PERIODS. SPECIFICALLY, THE LECTURE ON RADIATION PROTECTION WAS CONDUCTED WITH NO INSTRUCTOR AVAILABLE FOR 90% OF THE TIME. THE LECTURES ON GENERAL AND SPECIFIC OPERATING CHARACTERISTICS WERE CONDUCTED WITH NO INSTRUCTOR AVAILABLE FOR 50% OF THE TIME. AND, THE LECTURE ON INSTRUMENTATION WAS CONDUCTED WITH NO INSTRUCTOR AVAILABLE FOR 20% OF THE TIME. (7922 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

UNIT IN NORMAL POWER OPERATION.

+ LAST IE SITE INSPECTION DATE: MARCH 4-7, 1980

+ INSPECTION REPORT NO: 50-325/80-07

SUMMARY

INSPECTION SUMMARY

+ INSPECTION NOVEMBER 27-29 (79-42): THIS SPECIAL, ANNOUNCED INSPECTION INVOLVED 25 INSPECTOR-HOURS ON SITE IN THE AREA OF UNIT 1 REACTOR TRIP THAT OCCURRED ON NOVEMBER 20, 1979. OF THE AREA INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION FEBRUARY 5-8 (80-04): THIS SPECIAL, ANNOUNCED INSPECTION INVOLVED 29 INSPECTOR-HOURS ON SITE IN THE AREAS OF SMALL BREAK LOSS OF COOLANT PROCEDURES AND TRAINING. WITHIN THE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION JANUARY 2 - FEBRUARY 2 (80-05): THIS ROUTINE INSPECTION BY THE RESIDENT INSPECTOR INVOLVED 51 MANHOURS OF INSPECTION TIME ON SITE OF PREVIOUSLY IDENTIFIED EVENTS, PLANT OPERATIONS, RADIATION PROTECTION, PHYSICAL PROTECTION, QUALITY ASSURANCE, ORGANIZATION AND ADMINISTRATION, AND SURVEILLANCE TESTING. OF THE SIX AREAS INSPECTED NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION JANUARY 24 (80-06): THIS SPECIAL, ANNOUNCED INSPECTION INVOLVED 25 INSPECTOR-HOURS ONSITE FOR THE PURPOSE OF OBTAINING CAROLINA POWER AND LIGHT COMPANY MANAGEMENT AND OPERATING STAFF ASSESSMENT OF THE EFFECT ON PLANT SAFETY REGARDING IMPLEMENTATION OF THE NEAR TERM OPERATING LICENSE ACTION ITEMS PRESENTED IN NUREG 0660 (DRAFT). OF THE AREAS DISCUSSED, NO APPARENT ITEMS OF NONCOM- PLIANCE OR DEVIATIONS WERE IDENTIFIED. THE LICENSEE, BOTH MANAGEMENT AND OPERATING STAFF, PROVIDED SIGNIFICANT COMMENTS ON THE SCOPE AND IMPLEMENTATION DATES FOR THE NUMEROUS ITEMS IN THE DRAFT NUREG. SAFETY CONCERNS WERE EXPRESSED REGARDING OVER PRIORITIZING OF LICENSEES EFFORTS BY THE NRC AND THE HEAVY IMPACT ON LIMITED INDUSTRY RESOURCES.

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
80-20/ 03L-0	02/06/80	02/22/80	30 DAY LER	REACTOR ISOLATION PROTECTION VALVES FOUND CLOSED ON CONTAINMENT GAS AND PARTICULATE MONITORS.

INSPECTION MARCH 4-7 (80-07): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 24 INSPECTOR-HOURS ON SITE IN THE AREAS OF SPECIAL ULTRASONIC EXAMINATION OF RECIRCULATION SYSTEM INLET NOZZLES, (UNIT 2); INSERVICE INSPECTION OF CLASS 1 PIPING, (UNIT 2) PREPARATIONS FOR MARK I TORUS MODIFICATIONS, (UNITS 1 AND 2). OF THE THREE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

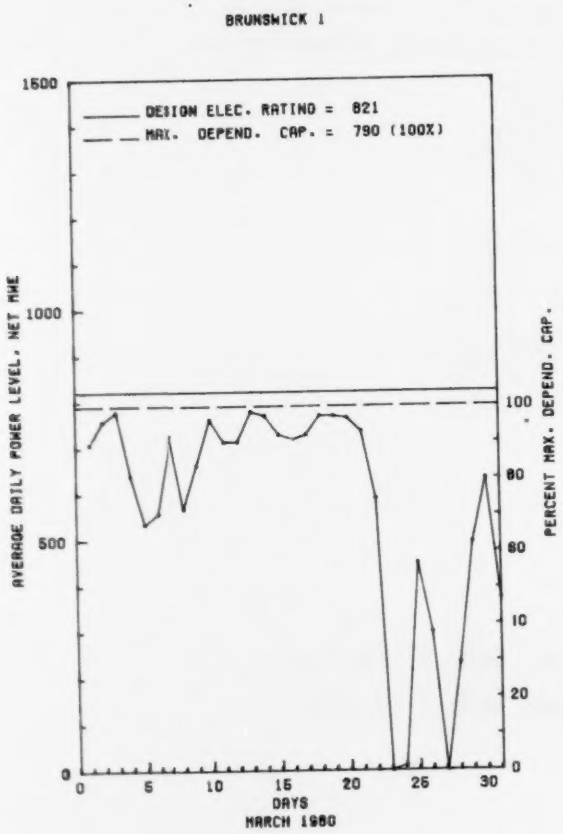
INSPECTION FEBRUARY 2-29 (80-08): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 30 INSPECTOR-HOURS ON SITE IN THE AREAS OF PLANT OPERATIONS, PLANT TOURS, RADIATION PROTECTION, QUALITY ASSURANCE, ORGANIZATION AND ADMINISTRATION, REVIEW OF LICENSEE RESPONSE TO IE BULLETINS AND FOLLOW- UP ON ABNORMAL EVENTS. OF THE SIX AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 5-12-80

REVIEWED BY NRR DOR: John N. Hannon **DATE:** 5-13-80

BRUNSWICK 1

AVERAGE POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-325
2. Utility Contact: E. Willis (919) 457-9521
3. Licensed Thermal Power (MWt): 2436
4. Nameplate Rating (Gross MWe): 963 x 0.9 = 867
5. Design Electrical Rating (Net MWe): 821
6. Maximum Dependable Capacity (Gross MWe): 815
7. Maximum Dependable Capacity (Net MWe): 790
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	2,184.0	26,641.0
12. Hours Reactor Was Critical	698.2	2,138.2	19,527.8
13. Reactor Reserve Shutdown Hours	.0	.0	1,647.1
14. Hours Generator On-Line	649.8	2,089.8	18,422.6
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,331,441	4,603,666	38,433,162
17. Gross Electrical Energy (MWH)	445,423	1,540,650	12,779,469
18. Net Electrical Energy (MWH)	430,293	1,489,315	12,297,125
19. Unit Service Factor	87.3	95.7	69.2
20. Unit Availability Factor	87.3	95.7	69.2
21. Unit Capacity Factor (MDC Net)	73.2	86.3	58.4
22. Unit Capacity Factor (DER Net)	70.4	83.1	56.2
23. Unit Forced Outage Rate	12.7	4.3	19.9

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
REFUELING 04/28/80
25. If Shut Down At End of Report Period, Estimated Date of Startup: 4/3/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
004	03/04/80	F	31.3	A	5		HC	HTEXCH	POWER WAS REDUCED TO LEAK CHECK THE CONDENSER WATERBOXES. 1B-NORTH CONDENSER WAS CHECKED FOR LEAKS BY HANGING HANDI-WRAP ON TUBE SHEETS. LEAKS COULD NOT BE IDENTIFIED BY THIS METHOD; A SCIENCE APPLICATIONS, INC., REP. WAS CONTACTED TO COME IN AND USE HELIUM TO FIND THE LEAKS. WATERBOX WAS RETURNED TO SERVICE AND LEAKS CONTROLLED WITH SAWDUST UNTIL THE VENDOR COULD GET ON SITE TO IDENTIFY LEAKS.
005	03/08/80	F	23.7	A	5		HC	HTEXCH	POWER REDUCED TO LEAK CHECK AND REPAIR CONDENSER WATERBOX 1B-NORTH. 1B-NORTH CONDENSER WAS CHECKED FOR LEAKS USING A HELIUM LEAK DETECTION METHOD. APPROXIMATELY FIVE TUBES WERE IDENTIFIED AS LEAKING. THESE TUBES WERE PLUGGED AND THE CONDENSER RETURNED TO SATISFACTORY SERVICE.
006	03/23/80	F	43.6	H	3		IA	INSTRU	REACTOR SCRAM DUE TO TURBINE STOP VALVE FAST CLOSURE CAUSED BY A REACTOR HIGH WATER LEVEL DURING TRANSFER FROM 3-ELEMENT FEEDWATER CONTROL TO SINGLE ELEMENT CONTROL IN ORDER TO REPAIR A FAILED STEAM FLOW INSTRUMENT. THE OPERATOR NULLED INCORRECTLY THE FEEDWATER MASTER CONTROLLER PRIOR TO SHIFTING TO MANUAL CONTROL WHICH CAUSED THE VESSEL LEVEL TO DROP WHEN MANUAL WAS SELECTED.
007	03/26/80	F	39.8	B	1		CB	VALVEX	POWER REDUCED AND THE UNIT SEPARATED FROM GRID DUE TO EXCESSIVE DRYWELL FLOOR LEAKAGE. A NITROGEN INERTING PROBLEM DUE TO THE INOPERABILITY OF THE AUXILIARY BOILERS EXTENDED THE OUTAGE. TWO VALVES IN THE DRYWELL WERE IDENTIFIED AS THE CAUSE(S) OF THE LEAKAGE. BOTH VALVES WERE REPAIRED AND THE EXCESSIVE LEAKAGE STOPPED.
008	03/31/80	F	10.8	A	3		EB	ELECON	REACTOR SCRAM ON LOW WATER LEVEL DUE TO THE APPARENT LOSS OF 1D BUS.

SUMMARY

THE REACTOR SCRAMMED ON 03/23/80 WHEN A STUCK PEN RECORDER GAVE ERRONEOUS WATER LEVEL INDICATIONS, AND ON 03/31/80 DUE TO APPARENT LOSS OF THE 1D BUS. LOAD REDUCTIONS OCCURRED ON 03/04/80 AND 03/08/80 TO LEAD CHECK CONDENSER WATERBOXES. THE UNIT WAS TAKEN OFF-LINE ON 03/26/80 TO EXAMINE DRYWELL FLOOR LEAKAGE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

BRUNSWICK 2

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	North Carolina/Brunswick 3 miles N of Southport
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2436
5.	Design Electrical Rating (MWE Net)	821
6.	Date of Initial Criticality	March 20, 1975
7.	Date of Commercial Operation	November 3, 1975
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Cape Fear River

Utility & Contractor Information

10.	Licensee	Carolina Power and Light Company
11.	Corporate Address	411 Fayetteville Street Raleigh, North Carolina 27602
12.	Corporate Contact	J. A. Jones Executive Vice President
13.	Architect/Engineer	United Engineers & Constructors, Inc.
14.	Nuclear Steam Supply System	GE
15.	Constructor	Brown and Root, Inc.

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R.C. Lewis
18.	IE Senior Inspector	P.J. Kellogg
19.	IE Resident Inspector	J. E. Outzs (919) 457-9531
20.	Licensing Project Manager	J. Hannon
21.	Docket Number	50-324
22.	License Number/Date of Issue	DPR-62, 12/27/74
23.	Public Document Room	Southport-Brunswick County Library 109 W. Moore Street Southport, North Carolina 28461

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO TECHNICAL SPECIFICATION 3.5V THE REACTOR WAS RESTARTED 12/12/76 WITH A DEFECTIVE SHOCK SUPPRESSOR ON THE RHR SYSTEM.
(7704 INFRACTION)

CONTRARY TO 10CFR20.104(A), DURING THE PERIOD MAY 20, 1976 TO SEPTEMBER 30, 1976, A MINOR, EMPLOYED BY A LICENSEE CONTRACTOR, WAS EXPOSED TO RADIATION IN EXCESS OF QUARTERLY LIMITS FOR WHOLEBODY EXPOSURES.
(7705 INFRACTION)

CONTRARY TO APPENDIX B TECHNICAL SPECIFCATION 5.3.1, WRITTEN PROCEDURES WERE NOT PROVIDED FOR IMPLEMENTATION OF TECHNICAL SPECIFICATIONS 2.2.2 AND 3.2.2.
(7705 INFRACTION)

CONTRARY TO TECHNICAL SPECIFICATION 6.8.1, FIRE PROTECTION PROCEDURE FP 4, "PERIODIC TEST SCHEDULING", WAS NOT FOLLOWED IN PERFORMING MONTHLY INSPECTIONS OF FIRE EXTINGUISHERS IN THAT AT LEAST THREE EXTINGUISHERS IN UNIT 2 REACTOR BUILDING WERE NOT INSPECTED DURING THE MONTH OF MARCH, 1977.
(7706 DEFICIENCY)

AS REQUIRED BY TECHNICAL SPECIFICATION 6.8.1, PROCEDURES SHALL BE WRITTEN AND IMPLEMENTED. PROCEDURTI-200, APPENDIX A, REQUIRES FORMAL CLASSROOM LECTURES FOR LICENSEES SCORING LESS THAN 80% IN A SPECIFIC AREA OF AN ANNUAL EXAMINATION. PROCEDURE TI-200, APPENDIX A, WAS NOT IMPLEMENTED IN THAT OF EIGHT (8) LECTURE SERIES CONDUCTED IN 1978, FOUR (4) WERE CONDUCTED LARGELY OR IN PART AS SELF-STUDY PERIODS. SPECIFICALLY, THE LECTURE ON RADIATION PROTECTION WAS CONDUCTED WITH NO INSTRUCTOR AVAILABLE FOR 90% OF THE TIME. THE LECTURES ON GENERAL AND SPECIFIC OPERATING CHARACTERISTICS WERE CONDUCTED WITH NO INSTRUCTOR AVAILABLE FOR 50% OF THE TIME. AND, THE LECTURE ON INSTRUMENTATION WAS CONDUCTED WITH NO INSTRUCTOR AVAILABLE FOR 20% OF THE TIME.
(7922 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

- UNIT SHUTDOWN FOR REFUELING AND TORUS MODIFICATION.
- LAST IE SITE INSPECTION DATE: MARCH 4-7, 1980
- INSPECTION REPORT NO: 50-324/80-07

SUMMARY

INSPECTION SUMMARY

- INSPECTION FEBRUARY 5-8 (80-04): THIS SPECIAL, ANNOUNCED INSPECTION INVOLVED 29 INSPECTOR-HOURS ON SITE IN THE AREAS OF SMALL BREAK LOSS OF COOLANT PROCEDURES AND TRAINING. WITHIN THE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION JANUARY 2 - FEBRUARY 2 (80-05): THIS ROUTINE INSPECTION BY THE RESIDENT INSPECTOR INVOLVED 50 MANHOURS OF INSPECTION TIME ON SITE OF PREVIOUSLY IDENTIFIED EVENTS, PLANT OPERATIONS, RADIATION PROTECTION, PHYSICAL PROTECTION, QUALITY ASSURANCE, ORGANIZATION AND ADMINISTRATION, SURVEILLANCE TESTING, AND UNIT 2 NEW FUEL INSPECTION. OF THE SEVEN AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-96/ 03L-1	11/20/79	01/14/80	30 DAY LER	RODS 42-33 AND 26-07 GAVE OVER TRAVEL ALARM WHEN DRAWN TO NO TCH 48 DURING NORMAL STARTUP.
80-04/ 03L-0	01/27/80	02/25/80	30 DAY LER	CONTAINMENT ATMOSPHERE MONITOR CAC-AT-1263 SHUTDOWN DUE TO LOW FLOW.
80-10/ 03L-0	01/14/80	02/11/80	30 DAY LER	MINIMUM CRITICAL POWER RATIO WAS 1.281 FOR BUNDLE 43-28.
80-15/ 03L-0	02/01/80	02/28/80	30 DAY LER	SUPPRESSION POOL LEVEL INDICATORS CAC-LT-2601 AND 3342 INOPERABLE.
80-22/ 03L-0	02/06/80	02/22/80	30 DAY LER	REACTOR ISO. PROT. VALVES FOR CONTAINMENT PARTICULATE AND GAS MONITORS FOUND CLOSED.

INSPECTION JANUARY 24 (80-06): THIS SPECIAL, ANNOUNCED INSPECTION INVOLVED 25 INSPECTOR-HOURS ONSITE FOR THE PURPOSE OF OBTAINING CAROLINA POWER AND LIGHT COMPANY MANAGEMENT AND OPERATING STAFF ASSESSMENT OF THE EFFECT ON PLANT SAFETY REGARDING IMPLEMENTATION OF THE NEAR TERM OPERATING LICENSE ACTION ITEMS PRESENTED IN NUREG 0660 (DRAFT). OF THE AREAS DISCUSSED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED. THE LICENSEE, BOTH MANAGEMENT AND OPERATING STAFF, PROVIDED SIGNIFICANT COMMENTS ON THE SCOPE AND IMPLEMENTATION DATES FOR THE NUMEROUS ITEMS IN THE DRAFT NUREG. SAFETY CONCERNS WERE EXPRESSED REGARDING OVER PRIORITIZING OF LICENSEES EFFORTS BY THE NRC AND THE HEAVY IMPACT ON LIMITED INDUSTRY RESOURCES.

INSPECTION MARCH 4-7 (80-07): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 22 INSPECTOR-HOURS ON SITE IN THE AREAS OF SPECIAL ULTRASONIC EXAMINATION OF RECIRCULATION SYSTEM INLET NOZZLES, (UNIT 2); INSERVICE

INSPECTION OF CLASS 1 PIPING, (UNIT 2) PREPARATIONS FOR MARK I TORUS MODIFICATIONS, (UNITS 1 AND 2). OF THE THREE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

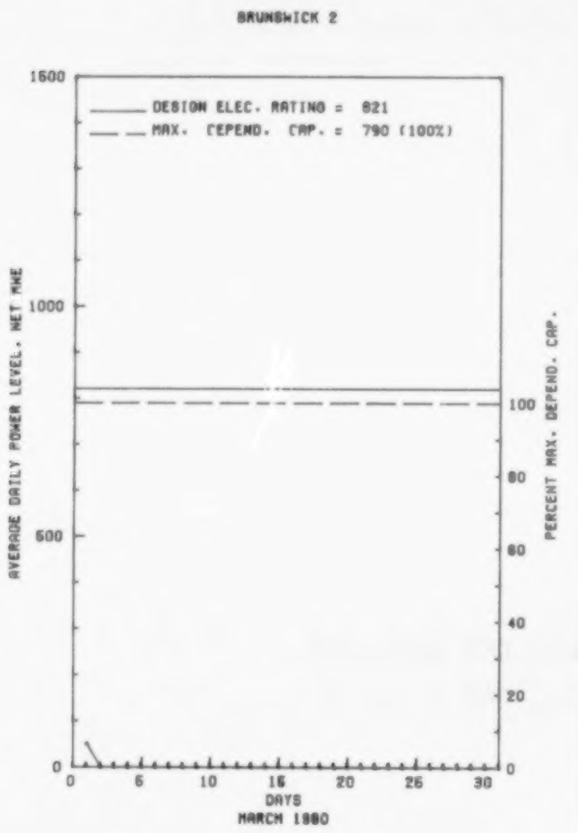
INSPECTION FEBRUARY 2-29 (80-08): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 25 INSPECTOR-HOURS ON SITE IN THE AREAS OF PLANT OPERATIONS, PLANT TOURS, RADIATION PROTECTION, QUALITY ASSURANCE, ORGANIZATION AND ADMINISTRATION, REVIEW OF LICENSEE RESPONSE TO IE BULLETINS AND FOLLOW-UP ON ABNORMAL EVENTS. OF THE SEVEN AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN SIX AREAS. IN ONE AREA, RADIATION PROTECTION, ONE APPARENT ITEM OF NONCOMPLIANCE WAS IDENTIFIED AS AN INFRACTION.

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 5-12-80

REVIEWED BY NRR DOR: John N Hannon **DATE:** 5-13-80

BRUNSWICK 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-324
2. Utility Contact: E. Willis (919) 457-9521
3. Licensed Thermal Power (MWt): 2436
4. Nameplate Rating (Gross MWe): 963 x 0.9 = 867
5. Design Electrical Rating (Net MWe): 821
6. Maximum Dependable Capacity (Gross MWe): 815
7. Maximum Dependable Capacity (Net MWe): 790
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	2,184.0	38,665.0
12. Hours Reactor Was Critical	2.7	1,365.7	26,689.6
13. Reactor Reserve Shutdown Hours	0	.0	.0
14. Hours Generator On-Line	2.7	1,228.1	25,102.1
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	4,471	2,604,960	46,339,422
17. Gross Electrical Energy (MWH)	1,503	872,613	15,487,687
18. Net Electrical Energy (MWH)	0	842,126	14,882,598
19. Unit Service Factor	.4	56.2	64.9
20. Unit Availability Factor	.4	56.2	64.9
21. Unit Capacity Factor (MDC Net)	.0	48.8	48.7
22. Unit Capacity Factor (DER Net)	.0	47.0	46.9
23. Unit Forced Outage Rate	.0	8.1	12.8

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 05/10/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	03/01/80	S	741.3	C	1		RC	FUELXX	REFUELING COMMENCED.

SUMMARY

THE UNIT BEGAN A REFUELING OUTAGE ON 03/01/80.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

CALVERT CLIFFS 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Maryland/Calvert 40 miles S of Annapolis
2.	Electric Reliability Council	Mid-Atlantic Area Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2700
5.	Design Electrical Rating (MWE Net)	846
6.	Date of Initial Criticality	October 7, 1974
7.	Date of Commercial Operation	May 8, 1975
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Chesapeake Bay

Utility & Contractor Information

10.	Licensee	Baltimore Gas and Electric Company
11.	Corporate Address	Gas and Electric Building Baltimore, Maryland 21203
12.	Corporate Contact	A. E. Lundvall, Jr. Vice President-Supply
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Combustion Engineering
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Senior Inspector	R. Keimig
19.	IE Resident Inspector	R. Architzel (301) 586-2626
20.	Licensing Project Manager	E. L. Conner, Jr.
21.	Docket Number	50-317
22.	License Number/Date of Issue	DPR-53, 7/31/74
23.	Public Document Room	Calvert County Library Prince Frederick, Maryland 20678

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO 10 CFR 50, APP B, CRITERION 16, CORRECTIVE ACTION WAS NOT PROMPTLY TAKEN TO CORRECT A GROUNDED CONDITION ON THE VITAL 125 VOLT D.C. BUS # 21. THIS GROUND WAS DISCOVERED ON 11/5/79 & NOT CORRECTED AS OF 1/29/80, AT WHICH TIME ANOTHER GROUND OCCURRED ON THE 12B REACTOR COOLANT PUMP CONTROL CIRCUIT & RESULTED IN AN INADVERTANT STARTING OF THE RCP.
(7923 INFRACTION)

CONTRARY TO 10 CFR 50, APP B, CRIT XVI, CORRECTIVE ACTION WAS NOT INITIATED TO DETERMINE THE CAUSE OF THE INADVERTANT OPENING OF A UNIT 1 POWER OPERATED RELIEF VALVE RV 404 WHICH OCCURRED ON 1/29/80 WHILE THE PLANT WAS IN COLD SHUTDOWN.
(7923 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS:

No significant information to report.

+ LAST IE SITE INSPECTION DATE: 4/7/80

+ INSPECTION REPORT NO: 50-317/80-05

SUMMARY

INSPECTION SUMMARY

+ 50-317/79-23 - Dec 26-28, Jan 2-4, 23-25 and 28-31: Routine inspection by the Resident Inspector (29 hours) of plant operations, the power distribution episode and review of reports. The NRC Resident Office was officially activated on January 31, 1980. Noncompliances: Two (Not investigating cause of PORV actuation; Failure to correct grounds on vital DC Bus 21).

NRC POWER RESTRICTIONS: NONE

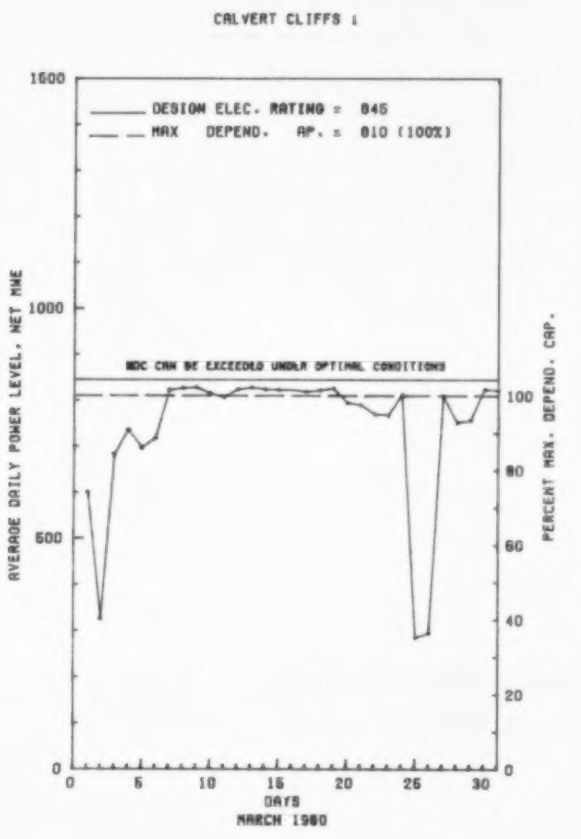
REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
80-009/03L	02/01/80	02/27/80	30 Day LER	Two fire hose stations in containment OOS on separate occasions due to hose removal
80-011/03L	02/08/80	03/07/80	30 Day LER	Shutdown cooling relief set improperly (required setpoint, 315 psig) due to procedure error and relieved during RCP venting operations at 250 psia
80-012/03L	02/11/80	03/03/80	30 Day LER	Control rod CEA-35 dropped into core from unknown cause during mode 2 ops at 1% power and was raised to group position in 18 min

REVIEWED BY OMPA LOEB: MR Beebe DATE: 05/12/80
REVIEWED BY NRR DOR: DATE: 5/12/80

CALVERT CLIFFS 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-317
2. Utility Contact: S. D. Merson (301) 234-5240
3. Licensed Thermal Power (MWt): 2700
4. Nameplate Rating (Gross MWe): 1020 x 0.9 = 918
5. Design Electrical Rating (Net MWe): 845
6. Maximum Dependable Capacity (Gross MWe): 845
7. Maximum Dependable Capacity (Net MWe): 810
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): 790
10. Reasons for Restrictions, If Any:
REDUCED TO 790 DUE TO TURBINE BLADE PROBLEMS.

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	2,184.0	42,949.0
12. Hours Reactor Was Critical	722.3	1,774.5	34,216.1
13. Reactor Reserve Shutdown Hours	7.6	7.6	1,068.1
14. Hours Generator On-Line	710.2	1,738.2	33,440.2
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,790,868	3,502,496	78,839,484
17. Gross Electrical Energy (MWH)	575,112	1,091,782	25,904,312
18. Net Electrical Energy (MWH)	549,264	1,027,267	24,676,473
19. Unit Service Factor	95.5	79.6	77.9
20. Unit Availability Factor	95.5	79.6	77.9
21. Unit Capacity Factor (MDC Net)	91.1	58.1	70.9
22. Unit Capacity Factor (DER Net)	87.4	55.7	68.0
23. Unit Forced Outage Rate	4.5	1.9	8.7

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-02	03/01/80	F	9.7	A	3		HF	PUMPXX	LEAK ON #14 CIRCULATING WATER PUMP COOLER IMPINGING ON THE INTAKE STRUCTURE HIGH-WATER LEVEL TRIP CIRCUITRY CAUSED LOSS OF ALL CIRCULATING WATER PUMPS.
80-03	03/25/80	F	24.1	A	3		RC	CONROD	LOSS OF VOLTAGE SUPPLY TO THE CONTROL RODS AND DRIVES.

SUMMARY

THE UNIT OPERATED THROUGHOUT THE REPORT PERIOD AT A 790 MWE (NET) LIMITATION DUE TO TURBINE BLADE PROBLEMS. LOAD WAS FURTHER REDUCED ON 03/01/80 FOR CONDUCTING CORE PHYSICS MEASUREMENTS, AND A REACTOR TRIP ALSO OCCURRED ON LOSS OF CONDENSER VACUUM WHEN THE CIRCULATING WATER PUMPS TRIPS AFTER A SALT WATER LEAK SPRAYED THE CIRCULATING WATER ROOM HIGH WATER LEVEL PROTECTIVE CIRCUITRY. THE REACTOR ALSO TRIPPED ON 03/25/80 DUE TO VOLTAGE INSTABILITY ON THE REACTOR TRIP BUS, AND TRIPPED AGAIN ON 03/26/80 DURING STARTUP. BRIEF LOAD REDUCTIONS WERE NECESSARY TO CHECK SALTWATEP LEAKAGE IN THE MAIN CONDENSER AND FOR MINOR MAINTENANCE.

LEGEND

[1] Type:
ed
duled

[2] Reason:
A–Equipment Failure
B–Maintenance or Test
C–Refueling
D–Regulatory Restriction
E–Operator Training & License Examination
F–Administrative
G–Operational Error
H–Other

[3] Method:
1–Manual
2–Manual Scram
3–Automatic Scram
4–Continuations
5–Load Reduction
9–Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

CALVERT CLIFFS 2

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Maryland/Calvert 40 miles S of Annapolis
2.	Electric Reliability Council	Mid-Atlantic Area Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2700
5.	Design Electrical Rating (MWE Net)	845
6.	Date of Initial Criticality	November 30, 1976
7.	Date of Commercial Operation	April 1, 1977
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Chesapeake Bay

Utility & Contractor Information

10.	Licensee	Baltimore Gas and Electric Company
11.	Corporate Address	Gas and Electric Building Baltimore, Maryland 21203
12.	Corporate Contact	A. E. Lundvall, Jr. Vice President-Supply
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Combustion Engineering
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	IE Senior Inspector	R. Keimig
19.	IE Resident Inspector	R. Architzel (301) 586-2626
20.	Licensing Project Manager	E. L. Conner, Jr.
21.	Docket Number	50-318
22.	License Number/Date of Issue	DPR-69, 8/13/76
23.	Public Document Room	Calvert County Library Prince Frederick, Maryland 20678

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO 10 CFR 50, APP B, CRIT 16, CORRECTIVE ACTION WAS NOT PROMPTLY TAKEN TO CORRECT A GROUNDED CONDITION ON THE VITAL 125 VOLT D.C. BUS #21 (SHARED BY UNITS 1 & 2). THIS GROUND WAS DISCOVERED ON 11/5/79 & NOT CORRECTED AS OF 1/29/80 AT WHICH TIME ANOTHER GROUND OCCURRED ON THE 12B REACTOR COOLANT PUMP CONTROL CIRCUIT RESULTING IN AN INADVERTANT STARTING OF THE RCP.
(7922 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS:

No significant information to report.

+ LAST IE SITE INSPECTION DATE: 4/7/80

+ INSPECTION REPORT NO: 50-318/80-05

SUMMARY

INSPECTION SUMMARY

+ 50-318/79-22 - Dec 26-28, Jan 2-4, 23-25 and 28-31: Routine inspection by the Resident Inspector (24 hours) of plant operations and review of reports. The NRC Resident Office was officially activated on January 31, 1980. Noncompliances: One (Failure to correct grounds on vital DC Bus 21).

NRC OWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

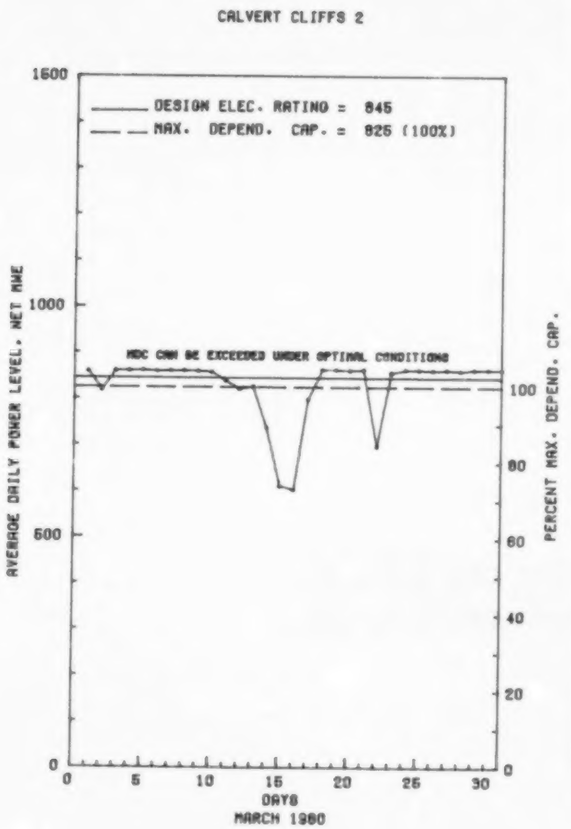
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
80-004/ 03L	01/21/80	02/19/80	30 Day LER	Channel B loss of load trip bypassed due to spurious tripping of channel
80-005/ 03L	01/22/80	02/22/80	30 Day LER	Channel B trip signals bypassed due to spurious high power level trips
80-006/ 03L	01/26/80	02/22/80	30 Day LER	Channel A RPS bypassed during cleanup of channel test panel
80-007/ 03L	01/30/80	02/29/80	30 Day LER	Control rod CEA-54 would not drive down electrically during surveillance. Problem cleared after 75 min
80-009/ 03L	01/31/80	02/29/80	30 Day LER	Azimuthal tilt exceeded TS limits when control rod CEA-21 dropped into core at high power due to unknown reason. Rod raised by operator 68 min later
80-011/ 01T	02/27/80	03/12/80	Other	EDG 21 did not start in 10 sec as rqrd and was left running to ensure availability until the 005 LPSI pump supplied by the other diesel was restored to service
80-012/ 03L	02/19/80	03/19/80	30 Day LER	Weld leak on charging pump common suction header relief valve, caused by vibration fatigue, temporarily repaired by encapsulation pending weld repair
80-014/ 03L	02/19/80	03/19/80	30 Day LER	CVCS "A" isolation logic CKT failure prevented channel isolation signal generation, necessitating module replacement

REVIEWED BY OMPA LOEB: DATE: 05/12/80

REVIEWED BY NRR DOR: DATE: 5/12/80

CALVERT CLIFFS 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-318
2. Utility Contact: S. D. Merson (301) 234-5240
3. Licensed Thermal Power (MWt): 2700
4. Nameplate Rating (Gross MWe): 1012 x 0.9 = 911
5. Design Electrical Rating (Net MWe): 845
6. Maximum Dependable Capacity (Gross MWe): 860
7. Maximum Dependable Capacity (Net MWe): 825
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	2,184.0	26,304.0
12. Hours Reactor Was Critical	744.0	1,964.9	21,909.8
13. Reactor Reserve Shutdown Hours	.0	.0	413.1
14. Hours Generator On-Line	744.0	1,956.7	21,565.2
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,906,192	5,035,835	53,280,066
17. Gross Electrical Energy (MWH)	643,786	1,691,733	17,696,956
18. Net Electrical Energy (MWH)	616,390	1,617,927	16,874,947
19. Unit Service Factor	100.0	89.6	82.0
20. Unit Availability Factor	100.0	89.6	82.0
21. Unit Capacity Factor (MDC Net)	100.4	89.8	77.8
22. Unit Capacity Factor (DER Net)	98.0	87.7	75.9
23. Unit Forced Outage Rate	.0	.0	6.3

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	NONE								

SUMMARY

OPERATION WAS AT NEAR FULL POWER DURING THE REPORT PERIOD, WITH MINOR MAINTENANCE ON THE INTAKE TRAVELING SCREENS AND THE STEAM GENERATOR FEEDPUMP.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

NRC POWER RESTRICTIONS: NONE

COOK 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Michigan/Berrien 11 miles S of Benton Harbor
2.	Electric Reliability Council	East Central Area Reliability Coordination Agreement
3.	Type of Reactor	PWR
4.	Power Level (MWT)	3250
5.	Design Electrical Rating (MWE Net)	1054
6.	Date of Initial Criticality	January 18, 1975
7.	Date of Commercial Operation	August 27, 1975
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10.	Licensee	AEP Corporation
11.	Corporate Address	Indiana & Michigan Power Co. 2 Broadway New York, New York 10004
12.	Corporate Contact	John Dolan Senior Executive V. P. Eng.
13.	Architect/Engineer	AEP
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Indiana & Michigan Power Co.

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Senior Inspector	D.C. Boyd
19.	IE Resident Inspector	R. Masse (616) 465-6775
20.	Licensing Project Manager	D. Wigginton
21.	Docket Number	50-315
22.	License Number/Date of Issue	DPR-58 issued 10/25/74
23.	Public Document Room	Maude Preston Palenske Memorial Library 500 Market Street St. Joseph, Michigan 49085

INSPECTION STATUS

ENFORCEMENT STATUS

PMI 2260 "STANDING ORDERS AND SPECIAL INSTRUCTIONS" REQUIRES PERSONNEL TO INITIAL AND DATE STANDING ORDERS, OR REQUIRES THAT SUPERVISORS INITIAL THAT SUBORDINATES HAVE HAD THE ORDER COMMUNICATED TO THEM. CONTRARY TO THIS REQUIREMENT, ON JANUARY 9, 1980, THE INSPECTOR OBSERVED THAT THE LATEST OSO'S (INCLUDING THOSE IMPLEMENTING NUREG-0578 RECOMMENDATIONS) HAD NOT BEEN REVIEWED BY UNIT 1 OR UNIT 2 LICENSED OPERATORS. TECHNICAL SPECIFICATION 6.8.1 REQUIRES IMPLEMENTATION OF PROCEDURES.
(8002 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

- Operating routinely.

LAST IE SITE INSPECTION DATE: December 17, 1979 and January 9-10, 1980

INSPECTION REPORT NO: 80-03

SUMMARY

INSPECTION SUMMARY

- No inspections conducted.

REPORTS RECEIVED FROM LICENSEE

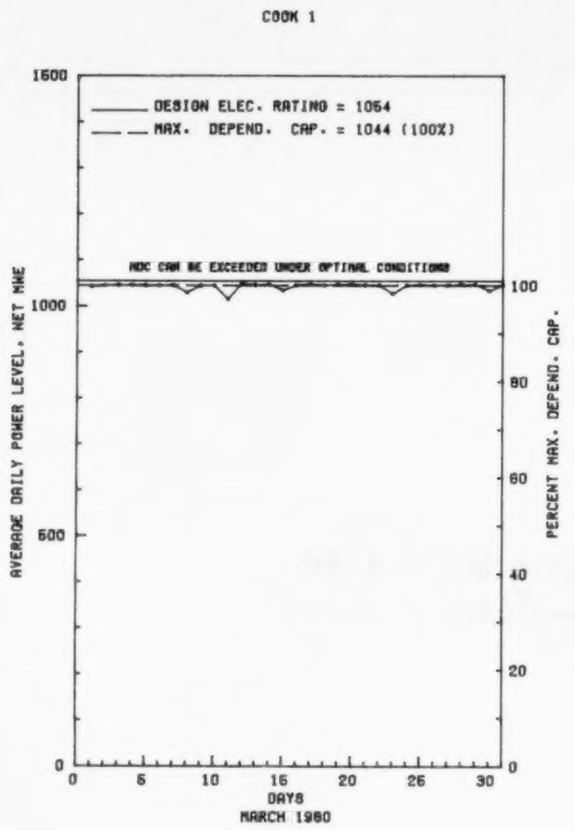
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
80-03/ 03L-0	02/19/80	03/11/80	30 Day LER	Found the spent fuel pit area monitor indicating erratically, and the high radiation alarm on.
80-09/ 03L-0	02/08/80	03/04/80	30 Day LER	During normal operation, steam generator No. 3 level indication became erratic and was declared inoperable. Thus not meeting the instrument requirements.

REVIEWED BY OMPA LOEB: T. C. Cintula DATE: 5·9·80

REVIEWED BY NRR DOR: D L Wigginton DATE: 5-12-80

COOK 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-315
2. Utility Contact: W. T. Gillett (616) 465-590
3. Licensed Thermal Power (MWt): 3250
4. Nameplate Rating (Gross MWe): 1280 x 0.9 = 1152
5. Design Electrical Rating (Net MWe): 1054
6. Maximum Dependable Capacity (Gross MWe): 1080
7. Maximum Dependable Capacity (Net MWe): 1044
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	[illegible],18[illegible].0	46,608.0
12. Hours Reactor Was Critical	744.0	1,786.8	34,748.5
13. Reactor Reserve Shutdown Hours	.0	.0	463.0
14. Hours Generator On-Line	744.0	1,780.6	37,863.8
15. Unit Reserve Shutdown Hours	.0	.0	321.0
16. Gross Thermal Energy (MWH)	2,410,76[illegible]	5,600,794	94,667,099
17. Gross Electrical Energy (MWH)	8[illegible]2,710	1,860,680	31,018,020
18. Net Electrical Energy (MWH)	77[illegible],515	1,795,577	29,792,891
19. Unit Service Factor	100.0	81.5	77.3
20. Unit Availability Factor	100.0	81.5	77.3
21. Unit Capacity Factor (MDC Net)	99.8	78.8	67.6
22. Unit Capacity Factor (DER Net)	98.9	78.0	62.9
23. Unit Forced Outage Rate	.0	18.5	7.6

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
NONE									

SUMMARY

OPERATION WAS AT NEAR FULL POWER DURING THE REPORT PERIOD, WITH MINOR REDUCTIONS FOR TURBINE VALVE TESTING AND PUMP MAINTENANCE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

COOK 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Michigan/Berrien 11 miles S of Benton Harbor
2.	Electric Reliability Council	East Central Area Reliability Coordination Agreement
3.	Type of Reactor	PWR
4.	Power Level (MWT)	3391
5.	Design Electrical Rating (MWE Net)	1100
6.	Date of Initial Criticality	March 10, 1978
7.	Date of Commercial Operation	July 1, 1978
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10.	Licensee	AEP Corporation Indiana & Michigan Power Co. Indiana & Michigan Electric Co.
11.	Corporate Address	2 Broadway New York, New York 10004
12.	Corporate Contact	John Dolan, VP/Chief Engineer (212) 422 - 4800
13.	Architect/Engineer	American Elec. Power Service Corp.
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Indiana & Michigan Power Co.

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Senior Inspector	D.C. Boyd
19.	IE Resident Inspector	R. Masse (616) 465-6775
20.	Licensing Project Manager	D. Wigginton
21.	Docket Number	50-316
22.	License Number/Date of Issue	DPR-74, 12/23/77
23.	Public Document Room	Maude Preston Palenske Memorial Library 500 Market Street St. Joseph, Michigan 49085

INSPECTION STATUS

ENFORCEMENT STATUS

PMI 2260 "STANDING ORDERS AND SPECIAL INSTRUCTIONS" REQUIRES PERSONNEL TO INITIAL AND DATE STANDING ORDERS, OR REQUIRES THAT SUPERVISORS INITIAL THAT SUBORDINATES HAVE HAD THE ORDER COMMUNICATED TO THEM. CONTRARY TO THIS REQUIREMENT, ON JANUARY 9, 1980, THE INSPECTOR OBSERVED THAT THE LATEST OSO'S (INCLUDING THOSE IMPLEMENTING NUREG-0578 RECOMMENDATIONS) HAD NOT BEEN REVIEWED BY UNIT 1 OR UNIT 2 LICENSED OPERATORS. TECHNICAL SPECIFICATION 6.8.1 REQUIRES IMPLEMENTATION OF PROCEDURES.
(8002 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

+ Operating routinely.

LAST IE SITE INSPECTION DATE: December 1, 1979 through January 11, 1980

INSPECTION REPORT NO: 80-02

SUMMARY

INSPECTION SUMMARY

+ No inspections conducted.

REPORTS RECEIVED FROM LICENSEE

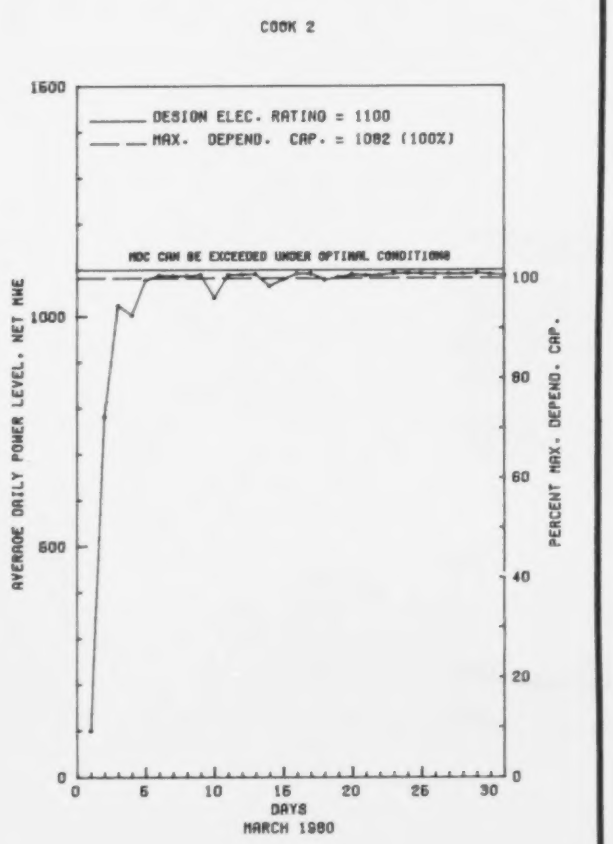
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
80-10/ 03L-0	02/16/80	03/17/80	30 Day LER	During surveillance test, suction valve VMO-102, for the containment equalization fan, HV-CEQ-2, failed to open as required.
80-11/ 03L-0	02/23/80	03/21/80	30 Day LER	While performing the weekly turbine stop and control valve testing a reduction in the reactor coolant system pressure occurred.

REVIEWED BY OMPA LOEB: T. C. Cintula **DATE:** 5.9.80

REVIEWED BY NRR DOR: D L Wigginton **DATE:** 5.12.80

COOK 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-316
2. Utility Contact: W. T. Gillett (614) 465-5901
3. Licensed Thermal Power (MWt): 3391
4. Nameplate Rating (Gross MWe): 1333 x 0.85 = 1133
5. Design Electrical Rating (Net MWe): 1100
6. Maximum Dependable Capacity (Gross MWe): 1118
7. Maximum Dependable Capacity (Net MWe): 1082
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	2,184.0	19,704.0
12. Hours Reactor Was Critical	728.4	1,729.8	12,809.7
13. Reactor Reserve Shutdown Hours	.0	.0	0
14. Hours Generator On-Line	726.1	1,647.1	12,149.0
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	2,418,799	5,168,837	37,393,217
17. Gross Electrical Energy (MWH)	801,060	1,698,480	11,852,890
18. Net Electrical Energy (MWH)	773,780	1,636,917	11,404,329
19. Unit Service Factor	97.6	75.4	70.5
20. Unit Availability Factor	97.6	75.4	70.5
21. Unit Capacity Factor (MDC Net)	96.1	69.3	64.5
22. Unit Capacity Factor (DER Net)	94.5	68.1	63.4
23. Unit Forced Outage Rate	2.4	4.0	12.3

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
74	03/01/80	F	17.9	A	3		CH	VALVOP	REACTOR/UNIT TRIP DUE TO EXTREME LOW LEVEL IN NO. 4 STEAM GENERATOR CAUSED BY PROBLEMS WITH THE FEEDWATER REGULATING VALVE FOR STEAM GENERATOR NO. 4. UNIT WAS RETURNED TO SERVICE THE SAME DAY. 100% REACTOR POWER WAS REACHED 03/02/80

SUMMARY

OPERATION WAS AT NEAR FULL POWER DURING THE REPORT PERIOD EXCEPT FOR A REACTOR/UNIT TRIP ON 03/01/80 WHEN LEVEL CONTROL IN THE NO. 4 STEAM GENERATOR BECAME SLUGGISH AFTER TURBINE CONTROL HAD BEEN LEFT IN THE MANUAL MODE FOLLOWING VALVE TESTING. ON 03/10/80, LOAD WAS SLIGHTLY REDUCED WHEN THE SOUTH HEATER DRAIN PUMP TRIPPED DUE TO A MALFUNCTION OF THE AUTOMATIC DISCHARGE VALVE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

COOPER STATION

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Nebraska/Nemaha 23 miles S of Nebraska City
2.	Electric Reliability Council	Mid-Continent Area Reliability Coordination Agreement
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2381
5.	Design Electrical Rating (MWE Net)	778
6.	Date of Initial Criticality	February 21, 1974
7.	Date of Commercial Operation	July 1, 1974 (50% rating)
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Missouri River

Utility & Contractor Information

10.	Licensee	Nebraska Public Power District
11.	Corporate Address	P. O. Box 499 Columbus, Nebraska 68601
12.	Corporate Contact	J. M. Pilant, Director Licensing and Quality Assurance
13.	Architect/Engineer	Burns and Roe, Inc.
14.	Nuclear Steam Supply System	GE
15.	Constructor	Burns and Roe, Inc.

Regulatory Information

16.	IE Region responsible	IV
17.	IE Branch Chief	G. L. Madsen
18.	IE Senior Inspector	T. F. Westerman
19.	IE Principal Inspector	G. L. Constable
20.	Licensing Project Manager	V. L. Rooney
21.	Docket Number	50-298
22.	License Number/Date of Issue	DPR-46, 1/18/74
23.	Public Document Room	Auburn Public Library 118 15th Street Auburn, Nebraska 68305

INSPECTION STATUS

ENFORCEMENT STATUS

TECHNICAL SPECIFICATION 6.3.3 REQUIRES THAT TEST PROCEDURES FOR PREVENTATIVE MAINTENANCE PROGRAMS. COOPER NUCLEAR STATION PROCEDURE 1.3 REQUIRES THAT PROCEDURES BE ADHERED TO. CONTRARY TO THE ABOVE, ON DECEMBER 31, 1979, (LER 80-01) OPERATORS FAILED TO PERFORM STEPS IN HPCI SURVEILLANCE PROCEDURE 6.2.2.3.12 THAT RESULTED IN THE HPCI AUXILIARY OIL PUMP BEING LEFT IN THE "PULL-TO-LOCK" POSITION FOR APPROXIMATELY 34 HOURS, RENDERING HPCI INOPERABLE FOR AUTOMATIC INITIATION DURING THAT PERIOD.
(8001 INFRACTION)

TECHNICAL SPECIFICATION 6.3.4 REQUIRES RADIATION CONTROL PROCEDURES TO BE MAINTAINED AND AVAILABLE. RADIATION PROTECTION AND CONTROL PROCEDURES 9.1.1.1, 9.1.2.1 AND 9.1.2.2 REQUIRE THAT CONTAMINATED AREAS BE POSTED WITH SIGNS INDICATING ENTRANCE REQUIREMENS AND THAT STATION EMPLOYEES ABIDE BY THESE PROCEDURAL REQUIREMENTS. ON MARCH 6, 1980, A SIGN ON THE HPCI ROOM DOOR INDICATED THAT PERSONS ENTERING THE AREA SHOULD WEAR PROTECTIVE SHOE COVERS AND GLOVES. CONTRARY TO THE ABOVE, THE NRC INSPECTOR OBSERVED A STATION OPERATOR IN THE HPCI ROOM WITHOUT THE REQUIRED PROTECTIVE CLOTHING.
(8001 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

CRACKS FOUND ON NO. 1 AND 2 LOW PRESSURE TURBINE ROTOR DISC.

FACILITY ITEMS (PLANS AND PROCEDURES):

MANAGERIAL ITEMS:

PLANT STATUS:

SHUT DOWN FOR REFUELING TORUS MODIFICATIONS AND INSPECTION OF WESTINGHOUSE TURBINE FOR CRACKING ON 3/1/80. OUTAGE EXPECTED TO EXTEND TO MAY 15, 1980.

LAST IE SITE INSPECTION DATE: JANUARY 29 - FEBRUARY 1, FEBRUARY 5-7, AND FEBRUARY 13-14, 1980

INSPECTION REPORT NO: 50-298/80-01

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
		3/1/80	ANNUAL REPORT FOR 1/1/79 - 12/31/79

SUMMARY

INSPECTION SUMMARY

INSPECTION CONDUCTED DECEMBER 3-14, 1979 (79-19): UNANNOUNCED INSPECTION OF WASTE SHIPMENT RECEIVED AT NUCLEAR ENGINEERING COMPANY, RICHLAND, WASHINGTON, DISPOSAL SITE. NO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED.

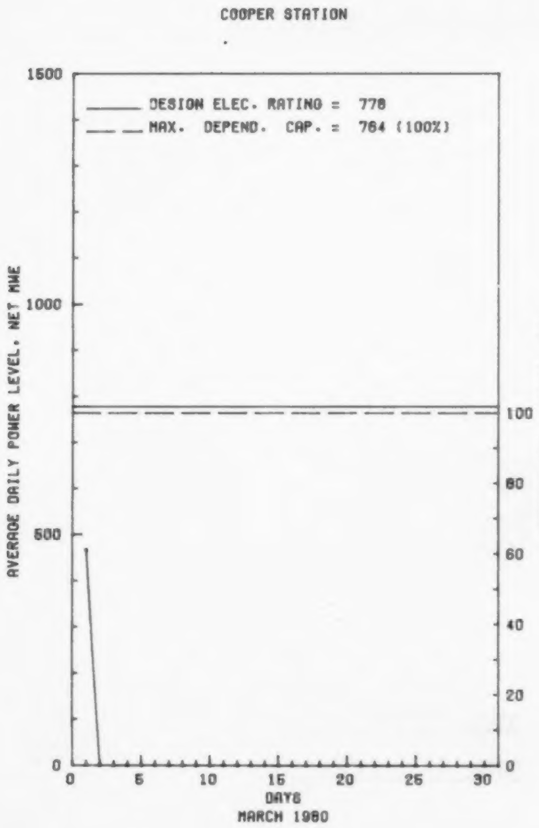
INSPECTION CONDUCTED JANUARY 29 - FEBRUARY 1, FEBRUARY 5-7, AND FEBRUARY 13-14, 1980 (80-01): UNANNOUNCED INSPECTION OF PLANT OPERATIONS, REVIEW OF NUREG-0578 IMPLEMENTATION, CONTAINMENT VENT AND PURGE, PREPARATIONS FOR REFUELING, NEUTRON MONITORING AND THERMAL LIMITS, SPENT FUEL STORAGE RACKS, SMALL BREAK LOCA PROCEDURES, FOLLOW-UP ON LER'S, IE BULLETINS, CIRCULARS, AND PREVIOUSLY IDENTIFIED UNRESOLVED ITEMS. THE INSPECTION INVOLVED 138 INSPECTION-HOURS ON-SITE BY FOUR (4) NRC INSPECTORS. NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE NOTED IN 8 OF THE 10 AREAS INSPECTED. TWO ITEMS OF NONCOMPLIANCE (INFRACTION - FAILURE TO ADHERE TO A SURVEILLANCE PROCEDURE; DEFICIENCY - FAILURE TO WEAR REQUIRED PROTECTIVE CLOTHING WERE NOTED).

INSPECTION CONDUCTED BY REGION V (REPORT NOS. 79-20 AND 80-05) ON DECEMBER 20, 1979 AT RICHLAND, WASHINGTON, AND JANUARY 21, 1980 AT BEATTY, NEVADA. ANNOUNCED INSPECTION OF THE RECEIPT OF WASTE SHIPMENTS AT WASTE DISPOSAL SITES. ONE ITEM OF NONCOMPLIANCE (CATEGORY IV - FAILURE TO HAVE PROPER MARKINGS ON THE OUTSIDE OF THE SHIPPING CASK) WAS IDENTIFIED.

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 5-12-80
REVIEWED BY NRR DOR: **DATE:** 5/12/80

COOPER STATION

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-298
2. Utility Contact: P. J. Borer (402) 825-3811
3. Licensed Thermal Power (MWt): 2381
4. Nameplate Rating (Gross MWe): 983 x 0.85 = 836
5. Design Electrical Rating (Net MWe): 778
6. Maximum Dependable Capacity (Gross MWe): 787
7. Maximum Dependable Capacity (Net MWe): 764
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	2,184.0	50,425.0
12. Hours Reactor Was Critical	22.0	1,447.0	42,684.3
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	22.0	1,437.7	41,913.3
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	39,504	3,077,520	79,260,246
17. Gross Electrical Energy (MWH)	11,639	1,018,195	25,557,043
18. Net Electrical Energy (MWH)	11,181	981,128	24,639,405
19. Unit Service Factor	3.0	65.8	83.1
20. Unit Availability Factor	3.0	65.8	83.1
21. Unit Capacity Factor (MDC Net)	2.0	58.8	64.0
22. Unit Capacity Factor (DER Net)	1.9	57.7	62.8
23. Unit Forced Outage Rate	.0	1.7	4.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 05/15/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-5	03/01/80	S	722.0	C	2				1980 REFUELING AND MAINTENANCE OUTAGE.

SUMMARY

THE UNIT BEGAN A REFUELING OUTAGE ON 03/01/80.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

CRYSTAL RIVER 3

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Florida/Citrus 7 miles NW of Crystal River
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2462
5.	Design Electrical Rating (MWE Net)	825
6.	Date of Initial Criticality	January 14, 1977
7.	Date of Commercial Operation	March 13, 1977
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Gulf of Mexico

Utility & Contractor Information

10.	Licensee	Florida Power Corporation
11.	Corporate Address	3201 34th Street, South St. Petersburg, Florida 33733
12.	Corporate Contact	J. Hancock
13.	Architect/Engineer	Gilbert Associates, Inc.
14.	Nuclear Steam Supply System	B&W
15.	Constructor	J. A. Jones Construction Co.

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. Lewis
18.	IE Senior Inspector	R. Martin
19.	IE Resident Inspector	T. F. Stetka
20.	Licensing Project Manager	P. B. Erickson
21.	Docket Number	50-302
22.	License Number/Date of Issue	DPR-72, 12/3/76
23.	Public Document Room	Crystal River Public Library 668 N.W. First Crystal River, Florida 32639

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

REFUELING SHUTDOWN.

LAST IE SITE INSPECTION DATE: MARCH 31 - APRIL 4, 1980

INSPECTION REPORT NO: 50-302/80-18

SUMMARY

INSPECTION SUMMARY

INSPECTION DECEMBER 10-14 (79-55): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 33 INSPECTOR-HOURS ON SITE IN THE AREAS OF REVIEW OF OPERATIONS, FOLLOWUP OF PREVIOUSLY IDENTIFIED ITEMS, AND FOLLOWUP OF IE BULLETINS. OF THE THREE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN TWO AREAS; ONE ITEM OF NONCOMPLIANCE WAS FOUND IN ONE AREA (IMPROPER MAINTENANCE OF HOUSEKEEPING REQUIREMENTS).

INSPECTION FEBRUARY 11-15 (80-06): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 38 INSPECTOR-HOURS ON SITE IN THE AREAS OF LICENSEE ACTIONS ON PREVIOUS INSPECTOR IDENTIFIED ITEMS, INFRACTIONS AND UNRESOLVED ITEMS, REVIEW OF PLANT OPERATIONS, REVIEW OF TEST AND MEASUREMENT EQUIPMENT AND PLANT TOUR. OF THE FOUR AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION FEBRUARY 19-22 (80-08): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 26 INSPECTOR-HOURS ON SITE IN THE AREAS OF PLANT OPERATIONS, REVIEW OF PREVIOUSLY IDENTIFIED INSPECTOR ITEMS, REVIEW OF IE NOTICES, PLANT HOUSEKEEPING, AND FUEL RECEIPT ACTIVITIES. OF THE FOUR AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION FEBRUARY 25-28 (80-11): INCLUDED REVIEW OF VITAL AREA ACCESS; COMMUNICATIONS; ALARM STATIONS; DETECTION AIDS; SECURITY ORGANIZATION. THE INSPECTION INVOLVED 35 INSPECTOR-HOURS ON SITE BY ONE NRC INSPECTOR, HOWEVER 25 INSPECTOR-HOURS WERE DEVOTED TO RESPONSE TO THE LICENSEE'S SIGNIFICANT EVENT OF FEBRUARY 26, 1980. THE INSPECTION WAS BEGUN DURING AN OFF-SHIFT PERIOD; THREE INSPECTION HOURS DEVOTED TO THE SECURITY INSPECTION WERE ACCOMPLISHED DURING OFF-SHIFT PERIODS. THE LICENSEE WAS FOUND TO BE IN COMPLIANCE WITH NRC REQUIREMENTS WITHIN ALL AREAS EXAMINED DURING THE INSPECTION.

INSPECTION FEBRUARY 26 - MARCH 7 (80-14): THIS SPECIAL INSPECTION INVOLVED APPROXIMATELY 1,000 INSPECTOR-HOURS ON SITE IN THE AREAS OF EVALUATION OF THE OPERATIONAL, MAINTENANCE, HEALTH PHYSICS, ENVIRONMENTAL, EMERGENCY PREPAREDNESS, MANAGEMENT CONTROL, AND RECOVERY ASPECTS OF THE NON NUCLEAR INSTRUMENTATION POWER LOSS AND REACTOR TRIP TRANSIENT INVOLVING THE FACILITY ON FEBRUARY 26, 1980. OF THE SEVEN AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN SIX AREAS; ONE ITEM OF NONCOMPLIANCE WAS FOUND IN ONE AREA (INFRACTION - FAILURE TO FOLLOW EMERGENCY PREPAREDNESS PROCEDURES EM-206 AND EM-207).

REPORTS RECEIVED FROM LICENSEE

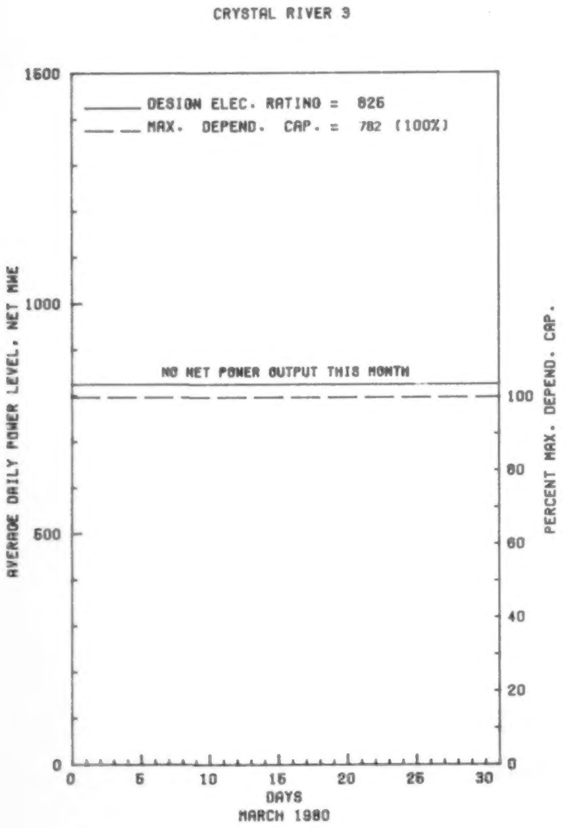
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-087/01T-5	02/18/80	03/03/80	14 DAY LER	REANALYSIS REQUIRED BY IE BULLETIN 79-14 SHOWS 30 PIPE SUPPORTS NEEDING MODIFICATION.
80-007/03L-0	01/30/80	02/28/80	30 DAY LER	AVERAGE REACTOR BUILDING TEMP OF 130.5 DEGREES FAHRENHEIT IN EXCESS OF TS 3.6.1.5 LIMIT OF 125 DEGREES FAHRENHEIT.
80-008/03L-0	02/06/80	03/04/80	30 DAY LER	DECAY HEAT PUMP DISCHARGE THROTTLE VALVE WOULD NOT CONTROL IN AUTOMATIC DUE TO LOOSE WIRES ON SELECTOR SWITCH.

REVIEWED BY OMPA LOEB: J E Bo **DATE:** 5/12/80

REVIEWED BY NRR DOR: P B Erickson **DATE:** 5/12/80

CRYSTAL RIVER 3

AVERAGE POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-302
2. Utility Contact: M.W. CULVER (904) 795-6486
3. Licensed Thermal Power (MWt): 2452
4. Nameplate Rating (Gross MWe): 989 x 0.9 = 890
5. Design Electrical Rating (Net MWe): 825
6. Maximum Dependable Capacity (Gross MWe): 821
7. Maximum Dependable Capacity (Net MWe): 782
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	2,184.0	26,760.0
12. Hours Reactor Was Critical	.0	1,305.5	16,455.7
13. Reactor Reserve Shutdown Hours	.0	.0	1,116.9
14. Hours Generator On-Line	.0	1,297.4	15,997.7
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	3,085,927	35,402,838
17. Gross Electrical Energy (MWH)	0	1,058,396	12,033,164
18. Net Electrical Energy (MWH)	0	1,009,335	11,400,975
19. Unit Service Factor	.0	59.4	59.8
20. Unit Availability Factor	.0	59.4	59.8
21. Unit Capacity Factor (MDC Net)	.0	59.1	54.5
22. Unit Capacity Factor (DER Net)	.0	56.0	51.6
23. Unit Forced Outage Rate	100.0	18.0	32.0

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 05/16/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-8	03/01/80	F	156.0	A	4	80-010/01	IF	INSTRU	CONTINUATION OF OUTAGE BEGAN 2/26/80 WITH LOSS OF POWER TO PART OF THE PLANT INSTRUMENTATION.
80-9	03/07/80	S	588.0	C	3		ZZ	ZZZZZZ	THE REFUELING OUTAGE WAS BEGUN EARLY SINCE THE PLANT WAS ALREADY SHUTDOWN FOR THE FORCED OUTAGE OF 2/26/80.

SUMMARY

THE UNIT REMAINED SHUTDOWN THROUGHOUT THE REPORT PERIOD FOR REFUELING. MAXIMUM DEPENDABLE CAPACITIES HAVE BEEN LOWERED TO 821 (GROSS) AND 782 (NET), BASED ON REVIEW OF PAST PERFORMANCE DATA.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

DAVIS-BESSE 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Ohio/Ottawa 21 miles E of Toledo
2.	Electric Reliability Council	East Central Area Reliability Coordination Agreement
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2772
5.	Design Electrical Rating (MWE Net)	906
6.	Date of Initial Criticality	August 12, 1977
7.	Date of Commercial Operation	11/21/77 (25%); 12/19/77 (40%) 01/23/78 (75%); 07/31/78 (100%)
8.	Condenser Cooling Method	Cooling Tower
9.	Condenser Cooling Water Source	Lake Erie

Utility & Contractor Information

10.	Licensee	Toledo Edison Co.
11.	Corporate Address	Edison Plaza, 300 Madison Avenue Toledo, Ohio 43652
12.	Corporate Contact	R. P. Crouse, V. P., Nuclear (419)259-5221
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	B&W
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Senior Inspector	R. F. Warnick
19.	IE Resident Inspector	L. Reyes (419) 898-2775
20.	Licensing Project Manager	D. Garner
21.	Docket Number	50-346
22.	License Number/Date of Issue	NPF-3, 4/22/77
23.	Public Document Room	Ida Rupp Public Library 310 Madison Street Port Clinton, OH 43452

INSPECTION STATUS

ENFORCEMENT STATUS

10 CFR 50, APPENDIX B, CRITERION VI, DOCUMENT CONTROL STATES THAT "MEASURES SHALL BE ESTABLISHED TO CONTROL THE ISSUANCE OF DOCUMENTS, SUCH AS INSTRUCTIONS, PROCEDURES AND DRAWINGS, INCLUDING CHANGES THERE TO, WHICH PRESCRIBE ALL ACTIVITIES AFFECTING QUALITY." ANSI N18.7 - 1972, PARAGRAPH 5, FACILITY ADMINISTRATIVE POLICIES AND PROCEDURES 2060 "DOCUMENT CONTROL" ALSO STATE THE REQUIREMENTS TO CONTROL THE ISSUANCE OF DOCUMENTS AND THE CHANGES TO BE INCORPORATED THERE TO. CONTRARY TO THE ABOVE, THE INSPECTOR FOUND THAT A NUMBER OF PLANT DRAWINGS WERE NOT BEING KEPT UP-TO-DATE AND OBSOLETE DOCUMENTS ARE NOT BEING PROPERLY CONTROLLED.
(7930 INFRACTION)

10 CFR 50, APPENDIX B, CRITERION XVII STATES THAT RECORDS SHALL BE IDENTIFIABLE AND RETRIEVABLE AND THAT THE APPLICANT SHALL ESTABLISH REQUIREMENTS CONCERNING RECORD RETENTION, SUCH AS DURATION, LOCATION AND ASSIGNMENT OF RESPONSIBILITIES. ANSI N45.2.9 AND AD 1848.04 REQUIRE THAT MEASURES BE ESTABLISHED WHICH ENSURE THAT QUALITY ASSURANCE RECORDS ARE PROPERLY COLLECTED, STORED AND MAINTAINED. CONTRARY TO THE ABOVE, A LIMITED ACCESS LIST WAS NOT ESTABLISHED TO CONTROL ACCESS TO THE STATION CENTRAL FILES AS REQUIRED BY AD 1848.04 AND THE ALLOWABLE OPERATING TRANSIENT CYCLE LOG BOOK WAS NOT KEPT CURRENT.
(7930 DEFICIENCY)

TECHNICAL SPECIFICATIONS 6.5.2.7.E STATES THAT "THE CNRB SHALL REVIEW VIOLATIONS OF CODES, REGULATIONS ORDERS, TECHNICAL SPECIFICATIONS, LICENSE REQUIREMENTS, OR OF INTERNAL PROCEDURES OR INSTRUCTIONS HAVING NUCLEAR SAFETY SIGNIFICANCE. TECHNICAL SPECIFICATION 6.5.2.10.B STATES THAT "REPORTS OF REVIEWS ENCOMPASSED BY SECTION 6.5.2.7 SHALL BE PREPARED, APPROVED AND FORWARDED TO THE EXECUTIVE VICE PRESIDENT, OPERATIONS AND CNRB MEMBERS WITHIN 14 DAYS FOLLOWING COMPLETION OF REVIEW." CONTRARY TO THE ABOVE REQUIREMENTS, THE INSPECTOR FOUND THAT VIOLATIONS OF TECHNICAL SPECIFICATIONS AND REGULATIONS IDENTIFIED IN NRC INSPECTION REPORTS 50-346/79-02, 79-16, 79-19 AND 79-29 WERE NOT REVIEWED BY THE CNRB AS REQUIRED.
(7930 INFRACTION)

TECHNICAL SPECIFICATION 3.7.6.1 REQUIRES TWO INDEPENDENT CONTROL ROOM EMERGENCY VENTILATION SYSTEMS TO BE OPERABLE IN MODES 1, 2, 3 AND 4. THE SURVEILLANCE PORTION OF THAT SPECIFICATION (TS 4.7.6.1.E.3) REQUIRES A MAKEUP FLOW OF 300 CFM PLUS OR MINUS 10% WHEN SUPPLYING THE CONTROL ROOM WITH OUTSIDE AIR. LINEUP FOR CONTROL ROOM EMERGENCY VENTILATION SYSTEM NO. 1 REQUIRES THAT SWITCH HA5261A ON CABINET CDE11C BE IN THE REMOTE POSITION. CONTRARY TO THE ABOVE ON JANUARY 24, 1980, WHILE IN MODE 1, THE INSPECTOR FOUND SWITCH HA5261A ON CABINET CDE11C IN THE LOCAL POSITION. THIS SWITCH ALIGNMENT WOULD HAVE PREVENTED THE CONTROL ROOM EMERGENCY VENTILATION SYSTEM NO. 1 TO BE OPERATED FROM THE CONTROL ROOM TO PROVIDE A SUPPLY OF OUTSIDE AIR.
(8001 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

+ Power was administratively maintained at approximately 72% due to a degraded pump shaft seal on reactor coolant pump (RCP) 1-1 and as a cycle stretchout to meet the fuel burnup window for the April 10, 1980 refueling outage. On March 23, 1980 due to further degradations of RCP 1-1 shaft seals, the pump was shut down and operation continued with 3 RCP's. The reactor was manually tripped on March 27, when one group of safety rods dropped into the core due to a power supply failure. (Returned to operation on March 30, 1980)

+LAST IE SITE INSPECTION DATE: January 14-18, 1980

+ INSPECTION REPORT NO: 80-02

SUMMARY

INSPECTION SUMMARY

+ Inspection on January 2-4, 7-11, 14-18, 21-25, 29-31; February 1, (80-01): A routine, unannounced inspection of follow up on previous inspection findings, plant operations, physical protection (security organization, physical barriers, access control, communications), follow up of licensee event reports, follow up of IE Bulletins, follow up of IE Circulars and General Station Training. The inspection involved 185 inspector hours onsite by the Resident Inspector. Of the 10 areas inspected, no items of noncompliance or deviations were found in nine areas; one item of noncompliance was identified in the other area (improper switch lineup in the Control Room Emergency Ventilation System).

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
80-10/ 01T-0	02/14/80	02/26/80	14 Day LER	Design deficiency on two reactor coolant hot leg pipe whip restraints at top of reactor coolant system hot leg.
80-13/ 03L-0	02/08/80	03/06/80	30 Day LER	Control rod absolute position indication was not responding.
80-14/ 03L-0	02/08/80	03/06/80	30 Day LER	Pressurizer sample system containment isolation valve would not close.
80-15/ 03L-0	02/10/80	03/07/80	30 Day LER	Control rod API declared inoperable due to asymmetric rod alarms.
80-18/ 03L-0	02/26/80	03/25/80	30 Day LER	BWST level indication reading higher than other channels.

+ Inspection on January 14-18, (80-02): Included a review of Security Plan; Security Program Audit; Security Organization; Protected Area Physical Barriers; Lighting; Access Control (Identification, Authorization and Badging); Access Control (Search); Access Control (Escorting); Access Control (Vital Control); Alarm Stations; Detection Aids; Assessment Aids; Communications; Testing and Maintenance; Compensatory Measures; Power Supply; Response; Locks, Keys, and Combinations; Records and Reports; Followup on Previous Items of Noncompliance; Followup on IE Bulletin No. 79-16. The inspection involved 133.5 inspector hours onsite by two NRC inspectors and one assistant. The inspection began during the day shift period; 39 inspection hours were accomplished during offshift periods. The licensee was found to be in compliance with NRC requirements within the 20 areas examined during this inspection except, for the following 9 items: (A) Access Control (Vital Areas) - Failure to adequately and effectively control access in accordance with security plan requirements; (B) Alarm Stations - Failure of the alarm stations to have adequate alarm monitoring capabilities; (C) Detection Aids - A portion of the active intrusion detection alarm system is not tamper-indicating; (D) Testing and Maintenance - The licensee failed to conduct proper tests of special purpose detectors; (E) Alarm Stations - Failure to provide an alarm subsystem in accordance with a security plan commitment; (F) Communications - Several redundant security communication systems were not properly secured; (G) Detection Aids - An inspector was able to circumvent intrusion detection sensors in certain areas; (H) Access Control (Identification Authorization, and Badging) - The licensee failed to log the entry of certain personnel according to procedure; (I) Access Control (Search) - The licensee failed to conduct adequate searches in accordance with security plan commitments.

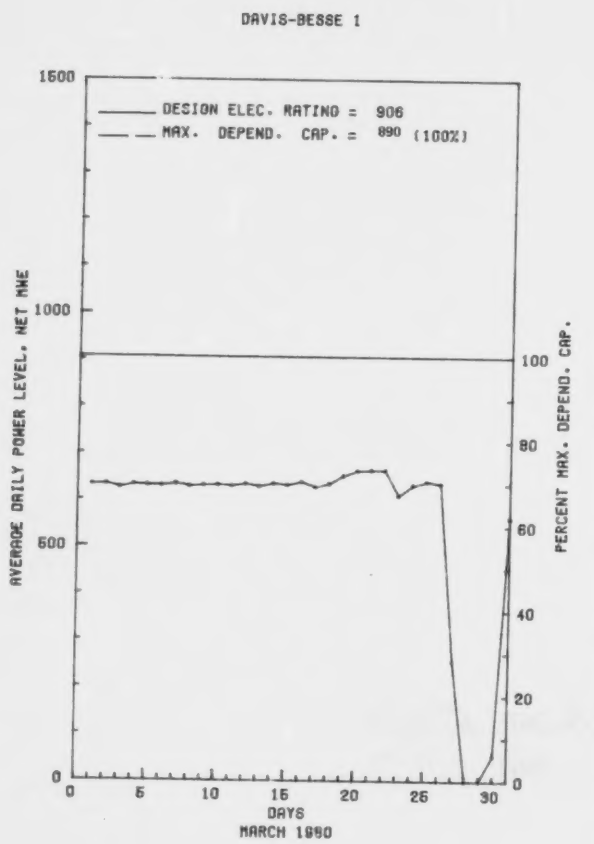
REVIEWED BY OMPA LOEB: P E Bobe **DATE:** 5/12/80

REVIEWED BY NRR DOR: **DATE:** 5/12/80

best copy available

DAVIS-BESSE 1

AVERAGE POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-346
2. Utility Contact: BILAL SARSOUR (419) 259-5000 X236
3. Licensed Thermal Power (MWt): 2772
4. Nameplate Rating (Gross MWe): 1069 x 0.9 = 962
5. Design Electrical Rating (Net MWe): 906
6. Maximum Dependable Capacity (Gross MWe): 934
7. Maximum Dependable Capacity (Net MWe): 890
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	2,184.0	22,709.0
12. Hours Reactor Was Critical	680.2	1,918.9	12,883.1
13. Reactor Reserve Shutdown Hours	.0	.0	2,875.8
14. Hours Generator On-Line	668.9	1,849.4	11,724.1
15. Unit Reserve Shutdown Hours	.0	.0	1,729.2
16. Gross Thermal Energy (MWH)	1,305,851	4,371,629	24,571,136
17. Gross Electrical Energy (MWH)	446,623	1,479,072	8,202,583
18. Net Electrical Energy (MWH)	416,323	1,389,440	7,560,018
19. Unit Service Factor	89.9	84.7	53.2
20. Unit Availability Factor	89.9	84.7	61.5
21. Unit Capacity Factor (MDC Net)	62.9	71.5	37.5
22. Unit Capacity Factor (DER Net)	61.8	70.2	36.9
23. Unit Forced Outage Rate	10.1	15.3	25.8

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
REFUELING - 04/10/80 - 12 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
2	03/27/80	F	75.1	A	2		RB	INSTRU	MANUAL REACTOR TRIP AFTER CONTROL ROD SAFETY GROUP 3 DROP.

SUMMARY

OPERATION WAS ROUTINE IN AND END-OF-CYCLE COASTDOWN. THE REACTOR WAS MANUALLY TRIPPED ON 03/27/80 AFTER CONTROL ROD SAFETY GROUP 3 DROPPED AND GROUP 4 BEGAN TO INSERT, BUT COULD NOT BE WITHDRAWN BY CONTROL ROOM OPERATOR.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

DRESDEN 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Illinois/Grundy 9 miles E of Morris
2.	Electric Reliability Council	Mid-America Interpool Network
3.	Type of Reactor	BWR
4.	Power Level (MWT)	700
5.	Design Electrical Rating (MWE Net)	200
6.	Date of Initial Criticality	October 15, 1959
7.	Date of Commercial Operation	July 4, 1960
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Kankakee River

Utility & Contractor Information

10.	Licensee	Commonwealth Edison Co.
11.	Corporate Address	P. O. Box 767 Chicago, Illinois 60690
12.	Corporate Contact	C. Reed
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	GE
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Senior Inspector	R. Spessard
19.	IE Resident Inspector	J. L. Barker (815)-942-9267
20.	Licensing Project Manager	S. J. Nowicki
21.	Docket Number	50-010
22.	License Number/Date of Issue	DPR-2 issued 9/28/59
23.	Public Document Room	Morris Public Library 604 Liberty Street Morris, Illinois 60451

INSPECTION STATUS

ENFORCEMENT STATUS

TECHNICAL SPECIFICATION 6.2 REQUIRES THAT LICENSEE PROCEDURES BE ADHERED TO. PROCEDURE DRP 1520-1, SECTION F.2, REQUIRES THAT A RADIOACTIVE MATERIAL SHIPMENT RECORD BE COMPLETED AS SPECIFIED IN APPENDIX I OF DRP 1520-01 FOR OFFSITE SHIPMENTS OF RADIOACTIVE MATERIAL. CONTRARY TO THE ABOVE, THE RADIOACTIVE MATERIAL SHIPMENT RECORD SPECIFIED IN APPENDIX I OF DRP 1520-01 WAS NOT COMPLETED FOR RADWASTE SHIPMENTS MADE BETWEEN JANUARY 1, 1979 AND NOVEMBER 28, 1979. (7923 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

+ Unit 1 shutdown on October 31, 1978 for an extended outage to upgrade ECCS, chemical cleaning and refueling. Outage expected to last a minimum of 30 months.

+LAST IE SITE INSPECTION DATE: January 7 through February 1, 1980

+ INSPECTION REPORT NO: 80-03

SUMMARY

INSPECTION SUMMARY

+ Inspection on January 7 through February 1, (80-03): Routine, unannounced resident inspection of maintenance activities; plant operations; physical protection - security organization, physical barriers, access control (identification, authorization, badging, search, and escorting) and communications; surveillance of safety-related systems/components required by technical specifications; IE Bulletin followup; IE Circular followup; review and audits; preparation for refueling; receipt of new fuel; and followup on headquarters requests. The inspection involved 132 inspector hours onsite by two NRC inspectors. No items of noncompliance were identified.

REPORTS RECEIVED FROM LICENSEE

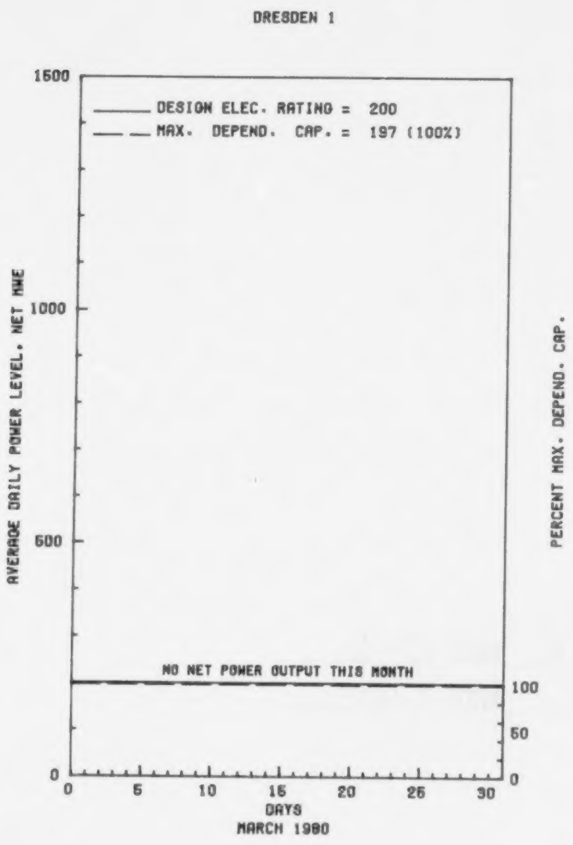
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
NONE				

REVIEWED BY OMPA LOEB: DATE: 5/12/80

REVIEWED BY NRR DOR: R Caruso DATE: 5/13/80

DRESDEN 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-010
2. Utility Contact: S. A. Cieszkiewicz (815) 942-2920
3. Licensed Thermal Power (MWt): 700
4. Nameplate Rating (Gross MWe): 245.5 x 0.85 = 209
5. Design Electrical Rating (Net MWe): 200
6. Maximum Dependable Capacity (Gross MWe): 205
7. Maximum Dependable Capacity (Net MWe): 197
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	2,184.0	17[illegible],984.0
12. Hours Reactor Was Critical	.0	.0	113,153.9
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	.0	.0	111,196.2
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	0	56,016,747
17. Gross Electrical Energy (MWH)	0	0	16,799,256
18. Net Electrical Energy (MWH)	-699	-2,332	15,807,495
19. Unit Service Factor	.0	.0	63.5
20. Unit Availability Factor	.0	.0	63.5
*21. Unit Capacity Factor (MDC Net)	.0	.0	45.9
22. Unit Capacity Factor (DER Net)	.0	.0	45.2
23. Unit Forced Outage Rate	.0	.0	11.2

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: JULY 1981

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
8	10/31/78	S	744.0	H	4				REFUEL AND CHEMICAL CLEANING OUTAGE CONTINUES.

SUMMARY

THE UNIT REMAINED SHUTDOWN THROUGHOUT THE REPORT PERIOD FOR REFUELING AND CHEMICAL CLEANING.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

NRC POWER RESTRICTIONS: NONE

DRESDEN 2

FACILITY DATA

Facility Description

1. Location (state/county) (nearest population center): Illinois/Grundy 9 miles E of Morris
2. Electric Reliability Council: Mid-America Interpool Network
3. Type of Reactor: BWR
4. Power Level (MWT): 2527
5. Design Electrical Rating (MWE Net): 794
6. Date of Initial Criticality: January 7, 1970
7. Date of Commercial Operation: JULY, 1970
8. Condenser Cooling Method: Cooling Lake
9. Condenser Cooling Water Source: Kankakee and Des Plaines Rivers

Utility & Contractor Information

10. Licensee: Commonwealth Edison Co.
11. Corporate Address: P. O. Box 767, Chicago, Illinois 60690
12. Corporate Contact: C. Reed
13. Architect/Engineer: Sargent & Lundy
14. Nuclear Steam Supply System: GE
15. Constructor: United Engineers & Constructors, Inc.

Regulatory Information

16. IE Region responsible: III
17. IE Branch Chief: R. Heishman
18. IE Senior Inspector: R. Spessard
19. IE Resident Inspector: J. L. Barker (815)-942-9267
20. Licensing Project Manager: S. J. Nowicki
21. Docket Number: 50-237
22. License Number/Date of Issue: DPR-19 issued 12/22/69
23. Public Document Room: Morris Public Library, 604 Liberty Street, Morris, Illinois 60451

INSPECTION STATUS

ENFORCEMENT STATUS

TECHNICAL SPECIFICATION 6.2 REQUIRES THAT LICENSEE PROCEDURES BE ADHERED TO. PROCEDURE DRP 1520-1, SECTION F.2, REQUIRES THAT A RADIOACTIVE MATERIAL SHIPMENT RECORD BE COMPLETED AS SPECIFIED IN APPENDIX I OF DRP 1520-01 FOR OFFSITE SHIPMENTS OF RADIOACTIVE MATERIAL. CONTRARY TO THE ABOVE, THE RADIOACTIVE MATERIAL SHIPMENT RECORD SPECIFIED IN APPENDIX I OF DRP 1520-01 WAS NOT COMPLETED FOR RADWASTE SHIPMENTS MADE BETWEEN JANUARY 1, 1979 AND NOVEMBER 28, 1979. (7927 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

Unit operating routinely.

+LAST IE SITE INSPECTION DATE: January 7 through February 1, 1980

+ INSPECTION REPORT NO: 80-03

SUMMARY

INSPECTION SUMMARY

+ Inspection on January 7 through February 1, (80-03): Routine, unannounced resident inspection of maintenance activities; plant operations; physical protection - security organization, physical barriers, access control (identification, authorization, badging, search, and escorting) and communications; surveillance of safety-related systems/components required by technical specifications; IE Bulletin followup; IE Circular followup; review and audits; preparation for refueling; receipt of new fuel; and followup on headquarters requests. The inspection involved 132 inspector hours onsite by two NRC inspectors. No items of noncompliance were identified.

REPORTS RECEIVED FROM LICENSEE

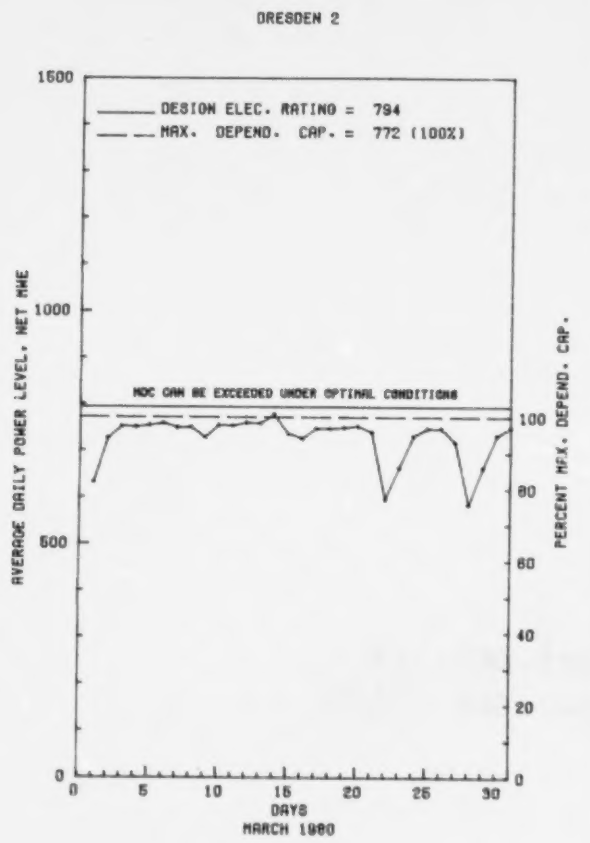
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
80-08/03L-0	01/26/80	02/25/80	30 Day LER	Unit 2/3 diesel generator inoperable due to heat exchanger tube leaks.

REVIEWED BY OMPA LOEB: DATE: 5/12/80

REVIEWED BY NRR DOR: DATE: 5/13/80

DRESDEN 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-237
2. Utility Contact: S. A. Cieszkiewicz (815) 942-2920 x263
3. Licensed Thermal Power (MWt): 2527
4. Nameplate Rating (Gross MWe): 920 x 0.9 = 828
5. Design Electrical Rating (Net MWe): 794
6. Maximum Dependable Capacity (Gross MWe): 812
7. Maximum Dependable Capacity (Net MWe): 772
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	2,184.0	86,640.0
12. Hours Reactor Was Critical	744.0	2,168.7	64,905.4
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	744.0	2,159.0	63,421.7
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,779,529	5,083,783	126,725,615
17. Gross Electrical Energy (MWH)	566,955	1,630,685	40,637,106
18. Net Electrical Energy (MWH)	539,620	1,551,569	38,439,681
19. Unit Service Factor	100.0	98.9	73.2
20. Unit Availability Factor	100.0	98.9	73.2
*21. Unit Capacity Factor (MDC Net)	94.0	92.0	57.5
22. Unit Capacity Factor (DER Net)	91.3	89.5	55.9
23. Unit Forced Outage Rate	.0	1.1	13.0

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	NONE								

SUMMARY

OPERATION WAS ROUTINE THROUGHOUT THE PERIOD, WITH NO OUTAGES OR REDUCTIONS REPORTED.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

NRC POWER RESTRICTIONS: NONE

DRESDEN 3

FACILITY DATA

Facility Description

1. Location (state/county) (nearest population center)	Illinois/Grundy 9 miles E of Morris
2. Electric Reliability Council	Mid-America Interpool Network
3. Type of Reactor	BWR
4. Power Level (MWT)	2527
5. Design Electrical Rating (MWE Net)	794
6. Date of Initial Criticality	January 31, 1971
7. Date of Commercial Operation	November 16, 1971
8. Condenser Cooling Method	Cooling Lake
9. Condenser Cooling Water Source	Kankakee and Des Plaines Rivers

Utility & Contractor Information

10. Licensee	Commonwealth Edison Co.
11. Corporate Address	P. O. Box 767 Chicago, Illinois 60690
12. Corporate Contact	D. L. Peoples
13. Architect/Engineer	Sargent & Lundy
14. Nuclear Steam Supply System	GE
15. Constructor	United Engineers & Constructors, Inc.

Regulatory Information

16. IE Region responsible	III
17. IE Branch Chief	R. Heishman
18. IE Senior Inspector	R. Spessard
19. IE Resident Inspector	J. L. Barker 815-942-9267
20. Licensing Project Manager	R. Bevan
21. Docket Number	50-249
22. License Number/Date of Issue	DPR-25 issued 1/12/71
23. Public Document Room	Morris Public Library 604 Liberty Street Morris, Illinois 60451

INSPECTION STATUS

ENFORCEMENT STATUS

TECHNICAL SPECIFICATION 6.2 REQUIRES THAT LICENSEE PROCEDURES BE ADHERED TO. PROCEDURE DRP 1520-1, SECTION F.2, REQUIRES THAT A RADIOACTIVE MATERIAL SHIPMENT RECORD BE COMPLETED AS SPECIFIED IN APPENDIX I OF DRP 1520-01 FOR OFFSITE SHIPMENTS OF RADIOACTIVE MATERIAL. CONTRARY TO THE ABOVE, THE RADIOACTIVE MATERIAL SHIPMENT RECORD SPECIFIED IN APPENDIX I OF DRP 1520-01 WAS NOT COMPLETED FOR RADWASTE SHIPMENTS MADE BETWEEN JANUARY 1, 1979 AND NOVEMBER 28, 1979. (7925 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

The unit was shut down on February 2, 1980 due to a failed jet pump. The licensee commenced a 12-week refueling/maintenance outage 6 weeks early as a result of this event. The outage was completed and the Unit returned to power April 21, 1980.

*LAST IE SITE INSPECTION DATE: January 7 through February 1, 1980

* INSPECTION REPORT NO: 80-03

SUMMARY

INSPECTION SUMMARY

* Inspection on January 7 through February 1, (80-03). Routine, unannounced resident inspection of maintenance activities; plant operations; physical protection - security organization, physical barriers, access control (identification, authorization, badging, search, and escorting) and communications; surveillance of safety-related systems/components required by technical specifications; IE Bulletin followup; IE Circular followup; review and audits; preparation for refueling; receipt of new fuel; and followup on headquarters requests. The inspection involved 132 inspector hours onsite by two NRC inspectors. No items of noncompliance were identified.

REPORTS RECEIVED FROM LICENSEE

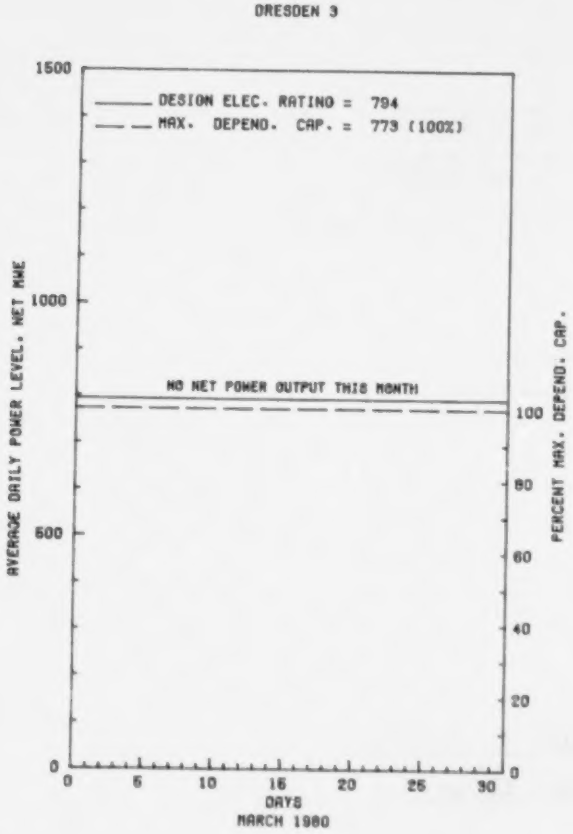
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
80-04/ 03L-0	02/02/80	02/28/80	30 Day LER	No. 13 jet pump failed.
80-05/ 01T-0	02/13/80	02/25/80	14 Day LER	Two MSIV scram limit switches found greater than 10%.

REVIEWED BY OMPA LOEB: DATE: 5/12/80

REVIEWED BY NRR DOR: RBBevan DATE: 5/13/80

DRESDEN 3

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-249
2. Utility Contact: S. A. Ciaszkiewicz (815) 942-2920 x263
3. Licensed Thermal Power (MWt): 2527
4. Nameplate Rating (Gross MWe): 920 x 0.9 = 828
5. Design Electrical Rating (Net MWe): 794
6. Maximum Dependable Capacity (Gross MWe): 812
7. Maximum Dependable Capacity (Net MWe): 773
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	2,184.0	76,225.0
12. Hours Reactor Was Critical	0	785.0	56,550.4
13. Reactor Reserve Shutdown Hours	0	0	0
14. Hours Generator On-Line	0	778.7	54,183.3
15. Unit Reserve Shutdown Hours	0	0	0
16. Gross Thermal Energy (MWH)	0	1,327,726	103,599,394
17. Gross Electrical Energy (MWH)	12	424,671	33,854,365
18. Net Electrical Energy (MWH)	-2,430	393,855	22,981,150
19. Unit Service Factor	0	35.7	71.3
20. Unit Availability Factor	0	35.7	71.0
21. Unit Capacity Factor (MDC Net)	.1	23.3	54.5
22. Unit Capacity Factor (DER Net)	0	22.7	53.0
23. Unit Forced Outage Rate	0	2.7	16.3

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 04/21/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
2	02/03/80	S	744.0	C	4		RC	FUELXX	REFUELING OUTAGE CONTINUES.

SUMMARY

THE UNIT REMAINED SHUTDOWN THROUGHOUT THE REPORT PERIOD FOR REFUELING. CORE RELOAD, JET PUMP REPAIR AND INSPECTION, AND MARK I MODIFICATIONS WERE COMPLETED.

LEGEND

[1] Type:
F–Forced
S–Scheduled

[2] Reason:
A–Equipment Failure
B–Maintenance or Test
C–Refueling
D–Regulatory Restriction
E–Operator Training & License Examination
F–Administrative
G–Operational Error
H–Other

[3] Method:
1–Manual
2–Manual Scram
3–Automatic Scram
4–Continuations
5–Load Reduction
9–Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

DUANE ARNOLD

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1. Location (state/county) (nearest population center)	Iowa/Linn 8 miles NW of Cedar Rapids
2. Electric Reliability Council	Mid-Continent Area Reliability Coordination Agreement
3. Type of Reactor	BWR
4. Power Level (MWT)	1658
5. Design Electrical Rating (MWE Net)	538
6. Date of Initial Criticality	March 23, 1974
7. Date of Commercial Operation	February 1, 1975
8. Condenser Cooling Method	Cooling Towers
9. Condenser Cooling Water Source	Cedar Rapids River Circulating Water System

Utility & Contractor Information

10. Licensee	Iowa Electric Light and Power
11. Corporate Address	IE Towers, P. O. Box 351 Cedar Rapids, Iowa 52406
12. Corporate Contact	D. Arnold, President
13. Architect/Engineer	Bechtel
14. Nuclear Steam Supply System	GE
15. Constructor	Bechtel

Regulatory Information

16. IE Region responsible	III
17. IE Branch Chief	R. Heishman
18. IE Senior Inspector	R. Spessard
19. IE Principal Inspector	G. Wright
20. Licensing Project Manager	T. Kevern
21. Docket Number	50-331
22. License Number/Date of Issue	DPR-49 issued 2/22/74
23. Public Document Room	Reference Service Cedar Rapids Public Library 428 Third Avenue, S.E. Cedar Rapids, Iowa 52401

INSPECTION STATUS

ENFORCEMENT STATUS

10 CFR, PART 50, APPENDIX B, CRITERION II STATES THAT "THIS PROGRAM SHALL BE DOCUMENTED BY WRITTEN POLICIES, PROCEDURES, OR INSTRUCTIONS AND SHALL BE CARRIED OUT THROUGHOUT PLANT LIFE IN ACCORDANCE WITH THOSE POLICIES, PROCEDURES." QAD 1301.3 REQUIRES MANAGERIAL REPORTING OF QA PROGRAM STATUS UTILIZING MONTHLY AUDIT STATUS REPORTS WHICH INCLUDE REPORTS ON ACTIVITIES HAVING SIGNIFICANT IMPACT ON PLANT QUALITY TRENDS. QAP 1405.2 REQUIRES THAT THE STATUS REPORTS INCLUDE MAJOR NONCONFORMANCES, CORRECTIVE ACTIONS, AND RESULTS OF AUDITS. CONTRARY TO THE ABOVE, A REVIEW OF STATUS REPORTS REVEALED THAT THE REPORTS DID NOT INCLUDE ACTIVITIES AND FINDINGS HAVING SIGNIFICANT IMPACT ON PLANT QUALITY TRENDS.
(7925 DEFICIENCY)

10 CFR, PART 50, APPENDIX B, CRITERION II STATES THAT "THIS (QUALITY ASSURANCE) PROGRAM SHALL BE DOCUMENTED BY WRITTEN POLICIES, PROCEDURES, OR INSTRUCTIONS AND SHALL BE CARRIED OUT THROUGHOUT PLANT LIFE IN ACCORDANCE WITH THOSE POLICIES, PROCEDURES, OR INSTRUCTIONS." ACD 1311.1 REQUIRES THAT THE SURVEILLANCE PROGRAM IDENTIFY ALL SURVEILLANCE TESTS AND CONTROL THEIR PERFORMANCE, EVALUATION OF RESULTS AND RECORDS. CONTRARY TO THE ABOVE, THE PROCEDURE COVERING SURVEILLANCE REQUIREMENT 4.6.A.1 IS NOT ADDRESSED, CONTROLLED OR TREATED AS A SURVEILLANCE REQUIREMENT AS REQUIRED BY ACD 1311.1
(7925 DEFICIENCY)

10 CFR PART 50, APPENDIX B, CRITERION II STATES THAT "THIS (QUALITY ASSURANCE) PROGRAM SHALL BE DOCUMENTED BY WRITTEN POLICIES, PROCEDURES, OR INSTRUCTIONS AND SHALL BE CARRIED OUT THROUGHOUT PLANT LIFE IN ACCORDANCE WITH THOSE POLICIES, PROCEDURES, OR INSTRUCTIONS." ADMINISTRATIVE CONTROL PROCEDURE (ACP) 1402.3 "PROCEDURE/INSTRUCTION REVIEW AND APPROVAL" REQUIRES THAT SPECIAL TEST PROCEDURES BE REVIEWED AND APPROVED BY SEVERAL PLANT PERSONNEL AT LEAST THE OPERATIONS COMMITTEE INCLUDING THE REACTOR AND PLANT PERFORMANCE ENGINEER. CONTRARY TO THE ABOVE, SPECIAL TEST PROCEDURE NO. 55, FEBRUARY 22, 1979, WAS NOT REVIEWED AND APPROVED BY THE REACTOR AND PLANT PERFORMANCE ENGINEER.
(7925 DEFICIENCY)

10 CFR, PART 50, APPENDIX B, CRITERION II STATES THAT "THE APPLICANT SHALL IDENTIFY THE STRUCTURES, SYSTEMS, AND COMPONENTS TO BE COVERED BY THE QUALITY ASSURANCE PROGRAM." QAD 1301.4, REV. 3, 4/2/79 STATES THAT "LISTS OF SAFETY RELATED STRUCTURES SYSTEMS AND COMPONENTS...SHALL BE PREPARED" AND THAT "A SYSTEM SHALL BE ESTABLISHED FOR DISTRIBUTION AND CONTROL OF THE SAFETY RELATED LIST." CONTRARY TO THE ABOVE, AN OFFICIAL SAFETY RELATED LIST HAD NOT BEEN ISSUED FOR USE AND PROCEDURES CONTROLLING THE SAFETY RELATED LIST DID NOT EXIST.
(7925 DEFICIENCY)

10 CFR 50, APPENDIX B, CRITERION V, STATES THAT ACTIVITIES AFFECTING QUALITY SHALL BE ACCOMPLISHED IN ACCORDANCE WITH INSTRUCTIONS, PROCEDURES, OR DRAWINGS. CONTRARY TO THE ABOVE, ACP 1401.4 AND 1404.5 WERE NOT ADHERED TO AS REQUIRED. (A) SAFETY-RELATED MAINTENANCE ACTIVITY REQUESTS WERE NOT INITIALED BY THE ORIGINATOR'S SUPERVISOR AS REQUIRED BY SECTION 6.3 OF ACP 1401.4 "CONTROL OF PLANT WORK." AND (B) HOLD-OFF TAG PLACEMENT ON TWO SAFETY-RELATED VALVES, DURING REPAIRS OF A LEAK ON THE HPCI/RCIC DRAIN TO THE MAIN CONDENSER, WAS NOT VERIFIED BY THE INDIVIDUAL OR RESPONSIBLE SUPERVISOR AS REQUIRED BY ACP 1404.5 "HOLD-OFF PROCEDURE."
(7925 INFRACTION)

10 CFR 50, APPENDIX B, CRITERION VI STATES THAT MEASURES SHALL ASSURE THAT DOCUMENTS SUCH AS DRAWINGS ARE DISTRIBUTED TO AND USED AT THE LOCATION WHERE THE PRESCRIBED ACTIVITY IS PERFORMED. QAP 1306.1, SECTION 5.6 REQUIRES THAT ALL DESIGN DOCUMENTS SHALL BE CONTROLLED AS NECESSARY TO PRECLUDE THE USE OF OUTDATED OR INAPPROPRIATE DOCUMENTS. CONTRARY TO THE ABOVE, TWO DRAWINGS IN THE CONTROL ROOM STAMPED "CONTROLLED DOCUMENT" WERE FOUND TO BE OUT OF DATE.
(7925 INFRACTION)

10 CFR 50.59 STATES THAT "THE LICENSEE SHALL ALSO MAINTAIN RECORDS OF TESTS AND EXPERIMENTS CARRIED OUT PURSUANT TO PARAGRAPH (A) OF THIS SECTION. THESE RECORDS SHALL INCLUDE A WRITTEN SAFETY EVALUATION WHICH PROVIDES THE BASIS FOR THE DETERMINATION THAT THE CHANGE, TEST OR EXPERIMENT DOES NOT INVOLVE AN UNREVIEWED SAFETY QUESTION." TECHNICAL SPECIFICATION 6.5.2.7 REQUIRES THE SAFETY COMMITTEE TO REVIEW: "A. THE SAFETY EVALUATIONS FOR (1) CHANGES TO PROCEDURES AND (2) TESTS OR EXPERIMENTS COMPLETED UNDER THE PROVISIONS OF SECTION 50.59, 10 CFR TO VERIFY THAT SUCH ACTIONS DID NOT CONSTITUTE AN UNREVIEWED SAFETY QUESTION... C. PROPOSED TESTS OR EXPERIMENTS WHICH INVOLVE AN UNREVIEWED SAFETY QUESTION AS DEFINED IN SECTION 50.59, 10 CFR." CONTRARY TO THE ABOVE, WHEN SPECIAL TEST PROCEDURE NO. 25 WAS REVIEWED BY THE OPERATIONS COMMITTEE AND FORWARDED TO THE SAFETY COMMITTEE, NEITHER THE MINUTES, NOR THE "SAFETY ANALYSIS" SECTION OF THE PROCEDURE RECORD ANY BASIS FOR A DETERMINATION NOR IS ANY CONCLUSION DOCUMENTED. MINUTES FROM SAFETY COMMITTEE MEETING NO. 115 ONLY STATE THAT IT WAS "REVIEWED AND APPROVED."
(7925 DEFICIENCY)

REPORTS RE[illegible]ED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-40/ 03L-0	10/10/79	02/22/80	30 Day LER	During daily surveillance testing the suppression chamber oxygen concentration was found to be 4.3%.
80-01/ 01T-0	01/24/80	02/07/80	14 Day LER	While performing daily surveillance on the containment atmospheric dilution system oxygen analyzers, AN-8181A sensing element was found to be flooded with apparent condensation from the sample line and was inoperable.
80-02/ 01T-0	02/06/80	02/20/80	14 Day LER	Fire watch removed with CO-2 system OOS.
80-04/ 01T-0	02/14/80	03/11/80	14 Day LER	Seat leakage on the inboard and combined seat and stem leakage on the outboard valves in excess of the 11.5 SCFH limit.
80-05/ 03L-0	02/05/80	03/06/80	30 Day LER	The RCIC system condensate pump would not pump down the RCIC turbine barometric condensor and pump space heaters tripped.
80-09/ 01T-0	03/04/80	03/18/80	14 Day LER	A section of split ring was lodged in impeller of main HPCI pump.

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

* The unit was shut down on February 9, 1980, to begin a 12-week refueling/maintenance outage.

LAST IE SITE INSPECTION DATE: October 9-12, 18-19, 25-26; November 1-2 and 20, 1979

INSPECTION REPORT NO: 79-25

SUMMARY

INSPECTION SUMMARY

* The following inspections have been conducted and will be reported at a later date:

December 10-13 (Report No. 79-32)

February 19-22 (Report No. 80-03)

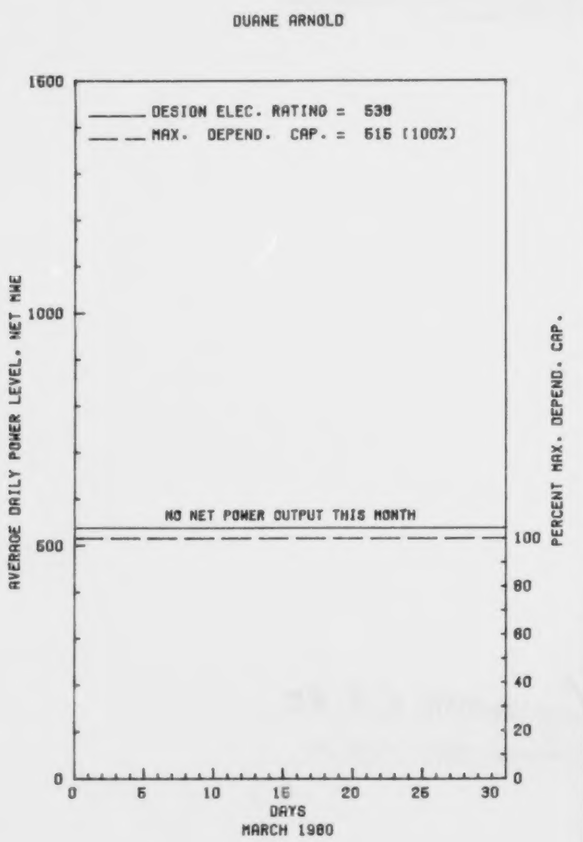
February 12-19/March 5-7 (Report No. 80-02)

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 5-12-80

REVIEWED BY NRR DOR: **DATE:** 5/13/80

DUANE ARNOLD

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-331
2. Utility Contact: J. Van Sickel (319) 851-5611
3. Licensed Thermal Power (MWt): 1658
4. Nameplate Rating (Gross MWe): 663 x 0.9 = 597
5. Design Electrical Rating (Net MWe): 538
6. Maximum Dependable Capacity (Gross MWe): 545
7. Maximum Dependable Capacity (Net MWe): 515
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	2,184.0	45,264.0
12. Hours Reactor Was Critical	.0	944.1	31,508.2
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	.0	944.1	30,763.2
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	1,219,968	38,349,528
17. Gross Electrical Energy (MWH)	0	428,336	12,852,702
18. Net Electrical Energy (MWH)	0	401,349	12,016,413
19. Unit Service Factor	.0	43.2	68.0
20. Unit Availability Factor	.0	43.2	68.0
21. Unit Capacity Factor (MDC Net)	.0	35.7	51.5
22. Unit Capacity Factor (DER Net)	.0	34.2	49.3
23. Unit Forced Outage Rate	.0	.0	20.4

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 04/14/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
1	02/09/80	S	744.0	C	4				CONTINUATION OF REFUELING AND MAINTENANCE OUTAGE.

SUMMARY

THE UNIT REMAINED SHUTDOWN THROUGHOUT THE REPORT PERIOD FOR REFUELING AND MAINTENANCE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

FARLEY 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Alabama/Houston 24 miles SE of Dothan
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2652
5.	Design Electrical Rating (MWE Net)	829
6.	Date of Initial Criticality	August 9, 1977
7.	Date of Commercial Operation	December 1, 1977
8.	Condenser Cooling Method	Once Through With Forced Draft Cooling Towers
9.	Condenser Cooling Water Source	Chattahoochee River

Utility & Contractor Information

10.	Licensee	Alabama Power Company
11.	Corporate Address	600 North 18th Street Birmingham, Alabama
12.	Corporate Contact	F. L. Clayton, Jr. Senior Vice President (205-323-5341)
13.	Architect/Engineer	Bechtel/SSI
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Daniel

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. Lewis
18.	IE Senior Inspector	R. D. Martin
19.	IE Resident Inspector	W. H. Bradford (205) 899-3386
20.	Licensing Project Manager	E. A. Reeves
21.	Docket Number	50-348
22.	License Number/Date of Issue	NPF-2, 6/25/77
23.	Public Document Room	G. S. Houston Memorial Library 212 W. Berdeshaw Street Dothan, Alabama 36301

INSPECTION STATUS

ENFORCEMENT STATUS

AS REQUIRED BY TECHNICAL SPECIFICATION 6.8.1.C WRITTEN PROCEDURES SHALL BE ESTABLISHED AND IMPLEMENTED FOR SURVEILLANCE AND TEST ACTIVITIES OF SAFETY RELATED EQUIPMENT. CONTRARY TO THE ABOVE ON JANUARY 27, 1978 WORK REQUEST NO 21908 ON BORIC AND TRANSFER PUMP 1-A WAS NOT COMPLETED IN ACCORDANCE WITH THE INSTRUCTIONS DELINEATED IN FARLEY NUCLEAR PLANT ADMINISTRATIVE PROCEDURE NO. 52 IN THAT THE 1-A BORIC ACID TRANSFER PUMP WAS ERRONEOUSLY INDICATED TO BE OPERABLE DURING THE PERFORMANCE OF MAINTENANCE AND THE RESTORATION AND RETEST REQUIREMENT WERE NOT LISTED OR APPENDED TO THE WORK REQUEST AS REQUIRED, EVEN THOUGH THE RETEST REQUIREMENTS HAD BEEN COMPLETED.
(8002 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

+ 100% POWER OPERATION.

+ LAST IE SITE INSPECTION DATE: MARCH 3-28, 1980

+ INSPECTION REPORT NO: 50-348/80-11

SUMMARY

INSPECTION SUMMARY

+ INSPECTION DECEMBER 31 - FEBRUARY 1 (80-02): THIS ROUTINE INSPECTION INVOLVED 50 INSPECTOR-HOURS ON SITE IN THE AREAS OF PLANT OPERATIONS, PREVIOUSLY IDENTIFIED ITEMS OF NONCOMPLIANCE, UNRESOLVED, AND OPEN ITEMS, IE CIRCULAR AND IE BULLETINS; LICENSEE EVENT REPORTS; UNIT 1 PLANT TOURS, AND PHYSICAL SECURITY. OF THE EIGHT AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN SEVEN AREAS; ONE APPARENT ITEM OF NONCOMPLIANCE WAS FOUND IN ONE AREA (DEFICIENCY INVOLVING FAILURE TO FOLLOW PROCEDURES IN COMPLETING EQUIPMENT TAGGING FORMS FOR MAINTENANCE).

INSPECTION FEBRUARY 26-29 (80-03): THIS SPECIAL, ANNOUNCED INSPECTION INVOLVED 29 INSPECTOR-HOURS ON SITE IN THE AREAS OF FIRE PROTECTION/PREVENTION. OF THE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION FEBRUARY 19-22 (80-05): THIS SPECIAL, ANNOUNCED INSPECTION INVOLVED 58 INSPECTOR-HOURS ON SITE IN THE AREAS OF SMALL BREAK LOSS OF COOLANT PROCEDURES AND TRAINING. WITHIN THE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

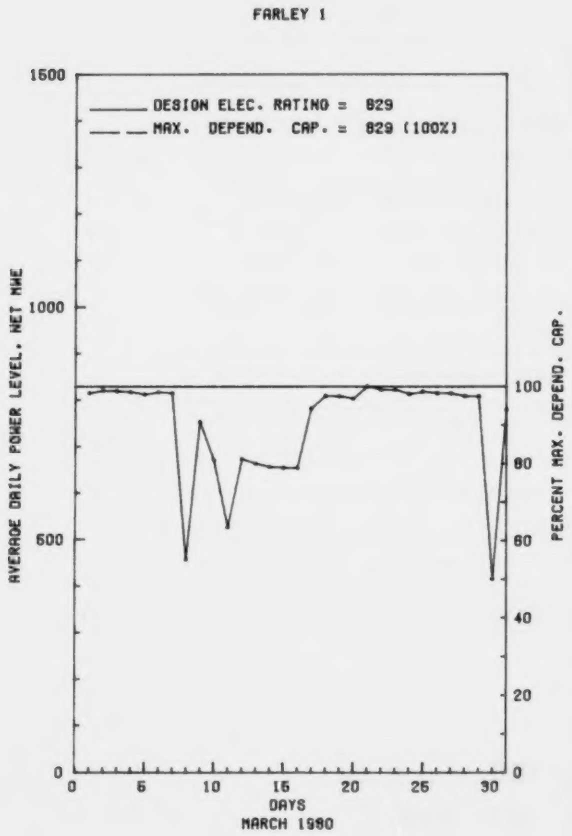
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
80-006/03L-0	01/26/80	02/25/80	30 DAY LER	CONTAINMENT AIR PARTICULATE MONITOR DECLARED INOPERABLE WHEN FILTER PAPER WOULD NOT ADVANCE DUE TO CARBON DUST FOULING FROM CARBON VANE VACUUM PUMP.
80-008/03L-0	02/03/80	02/25/80	30 DAY LER	COMPONENT COOLING WATER SNUBBER CW4-R121 DECLARED INOPERABLE WHEN RESERVOIR LINE DISCONNECTED SNUBBER PURGED/VENTED AND LINE RECONNECTED.

REVIEWED BY OMPA LOEB: T. C. Cintula **DATE:** 5-9-80

REVIEWED BY NRR DOR: Ed A. Reeves **DATE:** 5-13-80

FARLEY 1

AVERAGE POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-348
2. Utility Contact: W. G. Hairston,III (205) 899-5156
3. Licensed Thermal Power (MWt): 2652
4. Nameplate Rating (Gross MWe): 1045 x 0.85 = 888
5. Design Electrical Rating (Net MWe): 829
6. Maximum Dependable Capacity (Gross MWe): 860
7. Maximum Dependable Capacity (Net MWe): 829
8. If Changes Occur In Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	2,184.0	20,448.0
12. Hours Reactor Was Critical	727.6	1,747.6	12,683.1
13. Reactor Reserve Shutdown Hours	16.4	436.4	1,841.7
14. Hours Generator On-Line	721.2	1,684.7	12,275.2
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,800,910	4,161,514	30,759,529
17. Gross Electrical Energy (MWH)	582,412	1,345,586	9,869,624
18. Net Electrical Energy (MWH)	555,972	1,276,344	9,335,760
19. Unit Service Factor	96.9	77.1	60.0
20. Unit Availability Factor	96.9	77.1	60.0
21. Unit Capacity Factor (MDC Net)	90.1	70.5	55.1
22. Unit Capacity Factor (DER Net)	50.1	70.5	55.1
23. Unit Forced Outage Rate	3.1	7.1	7.5

24. Shutdowns Scheduled Over Next 6 Months (Type.Date.and Duration of Each): NONE
25. If Shut Down At End of Report Period. Estimated Date of Startup: h/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
021	03/08/80	F	5.9	H	2		HH	ZZZZZZ	UNIT MANUALLY TRIPPED DUE TO TRIP OF BOTH SGFP'S ON LOW SUCTION PRESSURE.
022	03/09/80	S	4.6	F	5		HH	ZZZZZZ	UNIT HELD AT 90% REACTOR POWER TO PERFORM A WALKDOWN OF SECONDARY SYSTEMS.
023	03/09/80	F	5.3	B	5		HH	ZZZZZZ	UNIT REDUCED AND HELD AT 90% REACTOR POWER FOR SECONDARY SIDE TROUBLE-SHOOTING AND MAINTENANCE.
024	03/10/80	F	6.2	F	5		HH	ZZZZZZ	UNIT REDUCED AND HELD AT 90% REACTOR POWER FOR SECONDARY SIDE TROUBLE-SHOOTING AND MAINTENANCE.
025	03/10/80	F	5.9	H	2		HH	ZZZZZZ	UNIT MANUALLY TRIPPED DUE TO TRIP OF BOTH SGFP'S ON LOW SUCTION PRESSURE.
026	03/11/80	F	23.6	F	5		HH	ZZZZZZ	UNIT REDUCED AND HELD AT 700 MWE FOR SECONDARY SIDE TROUBLE-SHOOTING AND MAINTENANCE.
027	03/12/80	F	2.6	F	5		HH	ZZZZZZ	UNIT REDUCED AND HELD AT 775 MWE FOR SECONDARY SIDE TROUBLE-SHOOTING AND MAINTENANCE.
028	03/12/80	F	22.8	F	5		HH	ZZZZZZ	UNIT REDUCED AND HELD AT 700 MWE FOR SECONDARY SIDE TROUBLE-SHOOTING AND MAINTENANCE.
029	03/14/80	F	49.8	F	5		HH	ZZZZZZ	UNIT HELD AT 90% REACTOR POWER FOR SECONDARY SIDE TROUBLE-SHOOTING AND MAINTENANCE.
030	03/20/80	S	4.5	B	5		HH	ZZZZZZ	UNIT REDUCED AND HELD AT 700 MWE TO PERFORM SECONDARY SIDE TESTING.
031	03/29/80	F	11.0	A	1	80-023/03L-0	SF	XXXXXX	UNIT TAKEN OFF LINE TO COMPLY WITH TECH. SPECS. 3.4.5.4. THE BORON INJECTION TANK WAS DECLARED INOPERABLE DUE TO NO RECIRCULATION FLOW.
032	03/31/80	F	5.5	H	5		HH	ZZZZZZ	UNIT REDUCED AND HELD AT 700 MWE FOR SECONDARY SIDE TROUBLE-SHOOTING AND MAINTENANCE.

SUMMARY

LOAD WAS REDUCED SEVERAL TIMES DURING THE PERIOD FOR SECONDARY SIDE TROUBLESHOOTING AND MAINTENANCE. THE UNIT WAS MANUALLY TRIPPED TWICE ON STEAM GENERATOR FEED PUMP LOW SUCTION PRESSURE, AND WAS TAKEN OFF LINE ON 03/29/80 DUE TO NO RECIRCULATION FLOW IN THE BORON INJECTION TANK.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

FITZPATRICK

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	New York/Oswego 8 miles NE of Oswego
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2436
5.	Design Electrical Rating (MWE Net)	821
6.	Date of Initial Criticality	November 17, 1974
7.	Date of Commercial Operation	July 28, 1975
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Lake Ontario

Utility & Contractor Information

10.	Licensee	Power Authority of the State of New York
11.	Corporate Address	10 Columbus Circle New York, New York 10019
12.	Corporate Contact	P. Lyon, Manager of Nuclear Operations
13.	Architect/Engineer	Stone & Webster
14.	Nuclear Steam Supply System	GE
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner (X248)
18.	IE Senior Inspector	E. McCabe (X251)
19.	IE Principal Inspector	T. F. Stetka (X276)
20.	Licensing Project Manager	P. Polk (X7872)
21.	Docket Number	50-333
22.	License Number/Date of Issue	DPR-59, 10/17/74
23.	Public Document Room	State University College of Oswego Penfield Library - Documents Oswego, NY 13126 (315)341-2323

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO ETS 2.2.3, AN UNCONTROLLED RELEASE OF REGENERATION WASTE WATER WITH AN ESTIMATED PH RANGE OF 4.0 TO 9.1 WAS DISCHARGED INTO THE DISCHARGE TUNNEL.
(8001 DEFICIENCY)

CONTRARY TO TS 3.3.A.2.A & 4.3.A.2.D, PARTIALLY INSERTED CONTROL ROD 18-35 WAS MADE INOPERABLE W/O EITHER FULLY INSERTING THE CONTROL ROD OR PERFORMING A SHUTDOWN MARGIN TEST PRIOR TO MAKING THE CONTROL ROD INOPERABLE.
(8001 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

An approximate 6 inch pipe crack has developed on the suction line to condensate booster pump B. A temporary fix has been made. The offgas system recombiner is out of service which has resulted in higher than normal stack releases.

FACILITY ITEMS (PLANS AND PROCEDURES):

+ Refueling outage scheduled to begin May 3, 1980.

MANAGERIAL ITEMS:

+ The Resident Manager position was filled on April 1, 1980. The Superintendent of Power position is presently being filled by qualified acting personnel until a final selection has been made.

PLANT STATUS:

+ The plant is operating at near full power with an average offgas rate at the SJAE of 10,000 microcuries per second and an average stack release rate of 2500 microcuries per second.

+ LAST IE SITE INSPECTION DATE: 4/11/80

+ INSPECTION REPORT NO: 50-333/80-05

SUMMARY

INSPECTION SUMMARY

+ None

REPORTS RECEIVED FROM LICENSEE

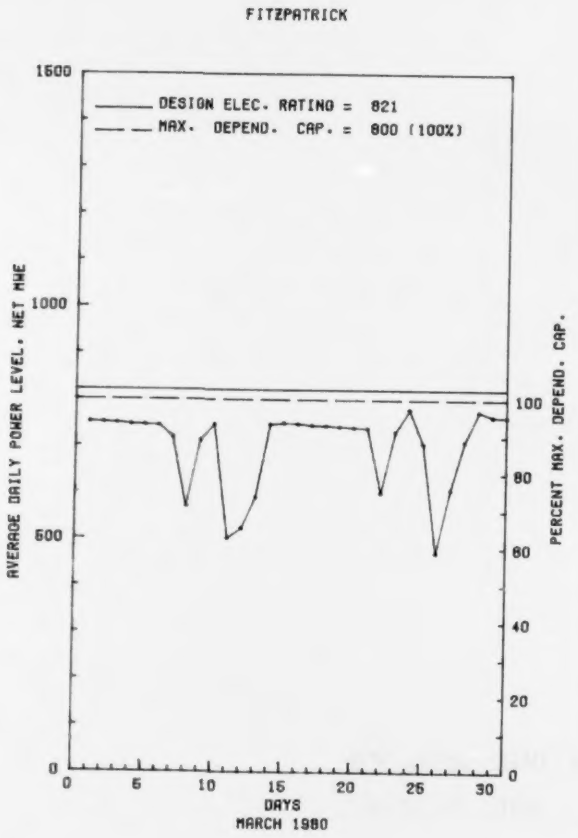
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
80-007/03L	01/09/80	02/08/80	30 Day LER	RHR loop B made inoperable due to pipe hanger failure
80-008/03L	01/09/80	02/08/80	30 Day LER	Core spray injection valve failed to open during surveillance testing due to torque and limit switch misadjustment
80-009/03L	01/10/80	02/08/80	30 Day LER	Diesel fire pump made inoperable to correct engine cooling system problem
80-010/03L	01/11/80	02/09/80	30 Day LER	Recirculation loop temperature instruments were found to be out of calibration due to personnel error in performing the previous calibration
80-011/03L	01/10/80	02/09/80	30 Day LER	Hudraulic snubber was found to be inoperative during surveillance testing. Snubber was replaced with a spare
80-012/03L	01/11/80	02/09/80	30 Day LER	Emergency diesel generator A tripped on low oil pressure during surveillance testing
80-013/03L	01/14/80	02/12/80	30 Day LER	Reactor lo-lo-lo level switch A did not operate properly during surveillance testing. Replaced with new switch
80-014/03L	01/16/80	02/12/80	30 Day LER	Torus water exceeded TS limits during HPCI testing. Level variance now permitted by TS during similar testing
80-015/03L	01/23/80	02/19/80	30 Day LER	Failure of LPCI independent power supply "A" inverter
80-016/03L	01/25/80	02/22/80	30 Day LER	Failure of drywell floor drain sump inboard isolation valve 20-MOV-82 due to a grounding situation caused by flooding the MOV terminal box
80-017/03L	01/30/80	02/25/80	30 Day LER	Drywell to torus differential pressure decreased to less than 1.7 PSID for about 1 hr. due to reactor building cooling
80-018/03L	02/01/80	02/25/80	30 Day LER	Core spray system pipe break detector out of calibration due to change in Tech Spec
80-019/03L	02/05/80	02/29/80	30 Day LER	Rod sequence control system did not come into service as required during plant shutdown due to calibration procedure error
80-020/04L	02/05/80	03/05/80	30 Day LER	Plant stack gas monitoring system inoperable due to freezing
80-021/03L	02/07/80	03/06/80	30 Day LER	Intermediate range monitor B made inoperable to repair a connector
80-022/03L	02/08/80	03/07/80	30 Day LER	Drywell atmosphere monitor A was made inoperable while B was out of service resulting in operation in a degraded mode
80-023/03L	02/11/80	03/11/80	30 Day LER	Reactor water chemistry sample for chlorides out of specification
80-024/03L	02/13/80	03/14/80	30 Day LER	HPCI steam isolation valve closed due to spurious isolation signal. Problem traced to loose wire on high area temperature thermocouple switch

REVIEWED BY OMPA LOEB: John J. Rizzo DATE: 5-12-80

REVIEWED BY NRR DOR: DATE: 5/9/80

FITZPATRICK

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-333
2. Utility Contact: E. Zufelt (315) 342-3840
3. Licensed Thermal Power (MWt): 2436
4. Nameplate Rating (Gross MWe): 981 x 0.9 = 883
5. Design Electrical Rating (Net MWe): 821
6. Maximum Dependable Capacity (Gross MWe): 830
7. Maximum Dependable Capacity (Net MWe): 800
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

ITEM 21 CALCULATED BY USING A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	2,184.0	41,017.0
12. Hours Reactor Was Critical	744.0	2,092.5	28,722.7
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	744.0	2,052.5	27,790.5
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,562,616	4,245,168	55,433,362
17. Gross Electrical Energy (MWH)	538,610	1,461,530	18,815,230
18. Net Electrical Energy (MWH)	521,005	1,414,405	18,204,980
19. Unit Service Factor	100.0	94.0	67.8
20. Unit Availability Factor	100.0	94.0	67.8
21. Unit Capacity Factor (MDC Net)	87.5	81.0	61.0
22. Unit Capacity Factor (DER Net)	85.3	78.9	54.1
23. Unit Forced Outage Rate	.0	6.0	20.1

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): SNUBBER INSPECTION & REFUELING - MAY 1980.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type¹	Duration (Hours)	Reason²	Method of Shutting Down Reactor³	Licensee Event Report No.	System Code⁴	Component Code⁵	Cause & Corrective Action to Prevent Recurrence
6	03/08/80	S	.0	H	5				POWER REDUCED FOR ROD PATTERN CHANGE
7	03/11/80	S	.0	H	5				CONDENSER TUBE LEAKS WITH RESULTING HIGH CONDUCTIVITY IN REACTOR VESSEL.
8	03/22/80	S	.0	H	5				POWER REDUCED FOR ROD PATTERN CHANGE.
9	03/22/80	S	.0	H	5				CONDENSER TUBE LEAKS WITH RESULTING HIGH CONDUCTIVITY IN REACTOR VESSEL.

SUMMARY

OPERATION WAS ROUTINE DURING THE REPORT PERIOD, EXCEPT FOR LOAD REDUCTIONS ON 03/07/80 AND 03/22/80 FOR CONTROL ROD ADJUSTMENT, AND 03/11/80 AND 03/26/80 TO REPAIR CONDENSER TUBE LEAKS.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

FORT CALHOUN

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Nebraska/Washington 19 miles N of Omaha
2.	Electric Reliability Council	Mid-Continent Area Reliability Coordination Agreement
3.	Type of Reactor	PWR
4.	Power Level (MWT)	1420
5.	Design Electrical Rating (MWE Net)	457
6.	Date of Initial Criticality	August 6, 1973
7.	Date of Commercial Operation	June 20, 1974
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Missouri River

Utility & Contractor Information

10.	Licensee	W. C. Jones Division Manager – Production Operations Omaha Public Power District
11.	Corporate Address	1623 Harney Street Omaha, Nebraska 68102
12.	Corporate Contact	T. E. Short Assistant General Manager
13.	Architect/Engineer	Gibbs & Hill, Inc.
14.	Nuclear Steam Supply System	Combustion Engineering
15.	Constructor	Gibbs & Hill, Durham & Richardson, Inc.

Regulatory Information

16.	IE Region responsible	IV
17.	IE Branch Chief	G. L. Madsen
18.	IE Senior Inspector	T. F. Westerman
19.	IE Principal Inspector	D. Kelley
20.	Licensing Project Manager	P. C. Wagner
21.	Docket Number	50-285
22.	License Number/Date of Issue	DPR-40, 5/24/73
23.	Public Document Room	W. Dale Clark Library 215 S. 15th Street Omaha, Nebraska 68102

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO 10 CFR 20.203(F), CONTAINERS CONTAINING SOLIDIFIED AND COMPACTED RADIOACTIVE WASTE, STORED IN A WORKING AREA OF THE AUXILLIARY BUILDING WERE NOT LABELED WITH A LABEL CLEARLY IDENTIFYING THEIR RADIOACTIVE CONTENTS AT THE TIME OF THE INSPECTION. THIS IS A DEFICIENCY.
(8004 DEFICIENCY)

CONTRARY TO TECHNICAL SPECIFICATION 5.11.2, VERY HIGH RADIATION AREAS WITHIN CONTAINMENT WERE (1) NOT BARRICADED AND CONSPICUOUSLY POSTED AND (2) PERSONNEL WERE ALLOWED ACCESS TO THESE AREAS WITHOUT A RADIATION MONITORING DEVICE OR UNDER THE POSITIVE CONTROL OF A PERSON QUALIFIED IN RADIATION PROTECTION PROCEDURES AT THE TIME OF INSPECTION. THIS IS AN INFRACTION.
(8004 INFRACTION)

CONTRARY TO TECHNICAL SPECIFICATION 5.11.1, HIGH RADIATION AREAS WITHIN CONTAINMENT WERE (1) NOT BARRICADED AND CONSPICUOUSLY POSTED AND (2) PERSONNEL WERE ALLOWED ACCESS TO THESE AREAS WITHOUT A RADIATION MONITORING DEVICE OR UNDER THE POSITIVE CONTROL OF A PERSON QUALIFIED IN RADIATION PROTECTION PROCEDURES AT THE TIME OF INSPECTION. THIS IS AN INFRACTION.
(8004 INFRACTION)

CONTRARY TO 10 CFR 19.11(A) OR (B), NEITHER THE DOCUMENTS REQUIRED BY 10 CFR 19.11(A) NOR THE NOTICE REQUIRED BY 10 CFR 19.11(B) WERE POSTED AT THE TIME OF INSPECTION. THIS IS A DEFICIENCY.
(8004 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

(1) Concrete shield wall at equipment hatch not installed per FSAR figure 11.2.2 (76-16) – Resolved by letter dated May 9, 1980.

FACILITY ITEMS (PLANS AND PROCEDURES):

MANAGERIAL ITEMS:

(1) Safety Audit and Review Committee review of Facility License Changes. (76-16)

PLANT STATUS:

No significant information

LAST IE SITE INSPECTION DATE: MARCH 24 - 27, 1980

INSPECTION REPORT NO: 50-285/80-07

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT

NONE

SUMMARY

INSPECTION SUMMARY

INSPECTION ON FEBRUARY 12-15, 1980 (80-04): ROUTINE, UNANNOUNCED INSPECTION OF THE LICENSEE'S RADIATION PROTECTION PROGRAM DURING REFUELING OPERATIONS INCLUDING HEALTH PHYSICS PROCEDURES; ADVANCED PLANNING; TRAINING; INTERNAL AND EXTERNAL EXPOSURE CONTROL; SURVEYS; CONTROL OF RADIOACTIVE MATERIAL; RESPIRATORY PROTECTION PROGRAM; AND A TOUR OF FACILITIES INVOLVED IN REFUELING. THE INSPECTOR ALSO INCLUDED AN INVESTIGATION INTO AN ALLEGATION OF DEFICIENCIES IN THE LICENSEE'S TRAINING PROGRAM. OF THE EIGHT (8) AREAS INSPECTED, FOUR ITEMS OF NONCOMPLIANCE WERE IDENTIFIED IN ONE AREA: (1) FAILURE TO BARRICADE AND CONTROL HIGH RADIATION AREAS, AN INFRACTION, (2) FAILURE TO BARRICADE AND CONTROL VERY HIGH RADIATION AREAS, AN INFRACTION, (3) FAILURE TO LABEL CONTAINERS CONTAINING RADIOACTIVE MATERIAL - A DEFICIENCY; (4) FAILURE TO POST DOCUMENTS REQUIRED BY 10 CFR 19.11, A DEFICIENCY. THREE OPEN ITEMS WERE IDENTIFIED DURING THE INSPECTION.

INSPECTION ON MARCH 5-7, 1980 (80-06): FACILITY ORGANIZATION AND OPERATION MEASUREMENTS AND CONTROLS, INVENTORY AND INVENTORY VERIFICATIONS, SHIPPING AND RECEIVING, STORAGE AND INVENTORY CONTROLS, RECORDS AND REPORTS AND MANAGEMENT OF MATERIAL CONTROL SYSTEM. THE LICENSEE WAS FOUND TO BE IN COMPLIANCE WITH NRC REQUIREMENTS IN THE SEVEN AREAS EXAMINED DURING THE INSPECTION.

INSPECTION ON FEBRUARY 12-15, 1980 (80-05): ROUTINE, UNANNOUNCED INSPECTION OF THE LICENSEE'S SECURITY ORGANIZATION. INCLUDED REVIEW OF SECURITY PLAN; SECURITY ORGANIZATION; PHYSICAL BARRIERS (PROTECTED AREA); PHYSICAL BARRIERS (VITAL AREAS); LIGHTING; ALARM STATIONS; DETECTION AIDS; ASSESSMENT AIDS; COMPENSATORY MEASURES; LOCKS, KEYS, AND COMBINATIONS; RECORDS AND REPORTS.

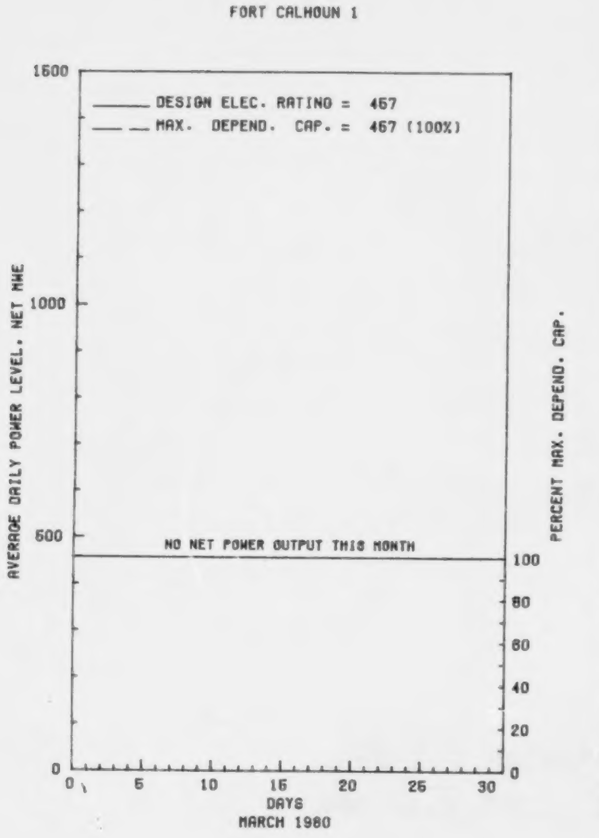
INSPECTION ON MARCH 24-27, 1980 (80-07): ROUTINE, UNANNOUNCED INSPECTION OF THE LICENSEE'S (1) RESPONSE TO OPEN AND UNRESOLVED ITEMS; (2) LICENSEE EVENT REPORTS; (3) RESPONSE TO INSPECTION AND ENFORCEMENT BULLETINS; (4) PROGRESS ON NUREG-0578; AND (5) REVIEW OF THE LICENSEE'S TURBINE INSPECTION. OF THE FIVE (5) AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

REVIEWED BY OMPA LOEB: MR Beebe **DATE:** 05/12/80

REVIEWED BY NRR DOR: PC Wagner **DATE:** 5/12/80

FORT CALHOUN

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-285
2. Utility Contact: B. J. Hickle (402) 536-4413
3. Licensed Thermal Power (MWt): 1420
4. Nameplate Rating (Gross MWe): 591 x 0.85 = 502
5. Design Electrical Rating (Net MWe): 457
6. Maximum Dependable Capacity (Gross MWe): 481
7. Maximum Dependable Capacity (Net MWe): 457
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Ne: MWe): NONE
10. Reasons for Restrictions, If Any: NONE

ITEM 21 CALCULATED BY USING A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	2,184.0	57,121.0
12. Hours Reactor Was Critical	.9	432.9	44,834.5
13. Reactor Reserve Shutdown Hours	.0	.0	1,309.5
14. Hours Generator On-Line	.0	430.5	43,881.6
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	574,731	53,809,758
17. Gross Electrical Energy (MWH)	0	190,604	17,850,834
18. Net Electrical Energy (MWH)	0	180,935	16,868,452
19. Unit Service Factor	.0	19.7	76.8
20. Unit Availability Factor	.0	19.7	76.8
*21. Unit Capacity Factor (MDC Net)	.0	18.1	65.1
22. Unit Capacity Factor (DER Net)	.0	18.1	64.6
23. Unit Forced Outage Rate	.0	.0	4.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 04/15/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-01	01/18/80	S	744.0	C	4				REACTOR SHUTDOWN FOR ANNUAL REFUELING & MAINTENANCE OUTAGE.

SUMMARY

THE UNIT REMAINED SHUTDOWN THROUGHOUT THE REPORT PERIOD FOR REFUELING AND MAINTENANCE.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B- Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

FORT ST. VRAIN

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Colorado/Weld 35 miles N of Denver (near Platteville)
2.	Electric Reliability Council	Western Systems Coordinating Council
3.	Type of Reactor	HTGR
4.	Power Level (MWT)	842
5.	Design Electrical Rating (MWE Net)	330
6.	Date of Initial Criticality	January 31, 1974
7.	Date of Commercial Operation	July 1, 1979
8.	Condenser Cooling Method	Towers
9.	Condenser Cooling Water Source	South Platte River/Shallow Wells

Utility & Contractor Information

10.	Licensee	Public Service Company of Colorado
11.	Corporate Address	P. O. Box 840 Denver, Colorado 80201
12.	Corporate Contact	C. K. Millen Senior Vice President
13.	Architect/Engineer	Sargent & Lundy
14.	Nuclear Steam Supply System	General Atomic Company
15.	Constructor	General Atomic Company

Regulatory Information

16.	IE Region responsible	IV
17.	IE Branch Chief	G. L. Madsen
18.	IE Senior Inspector	T. F. Westerman
19.	IE Resident Inspector	M. W. Dickerson (303) 785-2281
20.	Licensing Project Manager	G. Kuzmycz
21.	Docket Number	50-267
22.	License Number/Date of Issue	DPR-34, 12/21/73
23.	Public Document Room	Greeley Public Library City Complex Building Greeley, Colorado 80631

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO CRITERION VI OF APPENDIX B TO 10 CFR 50, THE LICENSEE PERFORMED TECHNICAL SPECIFICATION CALIBRATIONS USING OUTDATED PROCEDURE REVISIONS. EXAMPLE: PROCEDURE 5.4.1.3.2D-R WAS PERFORMED UNDER REVISION 13 ON 3/19/79 WHEN REVISION 15 WAS ISSUED EFFECTIVE 2/28/79.
(8002 INFRACTION)

TECH SPEC AC7.4 REQUIRES THAT WRITTEN PROCEDURES BE IMPLEMENTED. FSV ADMINISTRATIVE PROCEDURE ADM-13 REQUIRES THAT PROCEDURES BE ADHERED TO. IN PARTICULAR THAT TEST EQUIPMENT BE IDENTIFIED IN SECTION 3.0 OF SURVEILLANCE PROCEDURES. CONTRARY TO THE ABOVE, THE TEST CONDUCTOR FAILED TO IDENTIFY THE TEST EQUIPMENT USED DURING THE PERFORMANCE OF LINEAR POWER CHANNEL SCRAM CALIBRATION SURVEILLANCE TEST SR 5.4.1.1.4C-D FOR THE DATES OF JAN 5-8/80.
(8002 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

Areas related to potential equipment modification, which are under review and discussion with the NRC include: helium circulator auxiliaries; accident analysis relating to outlet temperature dispersion and orifice settings; hangers for 2" and smaller piping; and plant oscillations.

FACILITY ITEMS (PLANS AND PROCEDURES):

+ IMPLEMENTATION OF 10 CFR 73.55 PROVISIONS WERE IN EFFECT 3/21/80.

MANAGERIAL ITEMS:

PLANT STATUS:

+ PLANT STARTED POWER OPERATIONS.

LAST IE SITE INSPECTION DATE: FEBRUARY 27, - FEBRUARY 29, 1980 +

INSPECTION REPORT NO: 50-267/80-05 +

NRC POWER RESTRICTIONS: Restricted to 70% of Licensed Thermal Power.

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-06 03-L0	2/18/80	3/19/80	HYDRAULIC SNUBBER VSS-117 INOPERABLE - LCO 43.10
80-07 03-L0	2/19/80	3/18/80	TWO OF TWELVE HELIUM CIRCULATOR SEAL MALFUNCTION PRESSURE DIFFERENTIAL SWITCHES OUTSIDE LIMITS SPECIFIED BY LCO 4.4.1
80-08 03-L0	2/19/80	3/19/80	MOISTURE MONITOR 1121 LOW-LOW FLOW AND MOISTURE MONITOR 1121 LOW FLOW AND LOW-LOW FLOW ALARMS AT LESS THAN MINIMUM ACCEPTABLE LCO 4.4.1
80-09 01-T0	2/25/80	3/10/80	FIXED FEEDWATER FLOW, FIXED CIRCULATOR SPEED HIGH, AND PROGRAMMED CIRCULATOR SPEED/FEEDWATER FLOW TRIPS OUTSIDE TECHNICAL SPECIFICATION LCO 4.4.1
80-12 01-T0	3/11/80	3/25/80	TEMPERATURE LIMITS OF LCO 4.1.9 EXCEEDED FOR REACTOR ORIFICE VALVE SETTINGS AT UNEQUAL FLOW

SUMMARY

INSPECTION SUMMARY

+ INSPECTION FEBRUARY 1-29, 1980 (80-02): ROUTINE, ANNOUNCED INSPECTION OF EVENT REPORTS, CALIBRATION, SURVEILLANCE, BULLETINS/CIRCULARS, MAINTENANCE, PHYSICAL SECURITY, PLANT OPERATIONS, FOLLOWUP ON ITEMS OF NONCOMPLIANCE, FOLLOWUP ON INSPECTOR IDENTIFIED AND UNRESOLVED PROBLEMS, EMERGENCY PLANNING, CORE SAFETY LIMIT, AND REPORT REVIEWS. WITHIN THE TEN (10) AREAS INSPECTED, TWO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED (INFRACTION - FAILURE TO USE LATEST PROCEDURE REVISIONS WHILE PERFORMING CALIBRATION, AND DEFICIENCY - FAILURE TO PROVIDE A RECORD OF TEST EQUIPMENT).

INVESTIGATION ON JANUARY 28-30, 1980 (80-08): SPECIAL INVESTIGATION OF ALLEGED IRREGULAR SECURITY PRACTICES OF THE STANLEY SMITH SECURITY SYSTEM, INC., CONTRACT SERVICES. NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED. THE ALLEGATIONS WERE NOT SUBSTANTIATED.

INSPECTION ON FEBRUARY 11-12, 1980 (80-04): SCHEDULED ROUTINE MANAGEMENT MEETING TO REVIEW THE LICENSEE'S ENFORCEMENT HISTORY, DISCUSS REPORTABLE OCCURRENCES, REVIEW THE STATUS OF LICENSEE'S RESPONSES TO IE BULLETINS AND CIRCULARS, DISCUSS THE INSPECTION PROGRAM CONDUCTED BY THE OFFICE OF INSPECTION AND ENFORCEMENT, AND REVIEW CURRENT ITEMS OF CONCERN TO THE REGIONAL OFFICE STAFF. NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

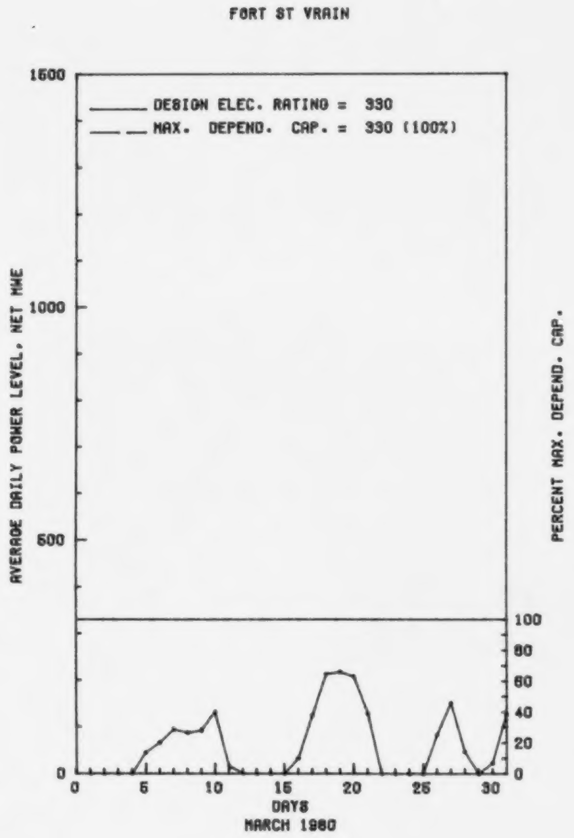
INSPECTION ON FEBRUARY 27-29, 1980 (80-05): ROUTINE, ANNOUNCED INSPECTION OF THE LICENSEE'S EMERGENCY PLANNING PROGRAM INCLUDING COORDINATION WITH OFFSITE AGENCIES, FACILITIES, SUPPLIES, EQUIPMENT, EMERGENCY PLAN AND PROCEDURES, TRAINING, TESTS, AND DRILLS AND INTERNAL AUDITS. THE INSPECTOR OBSERVED THE EMERGENCY DRILL ON FEBRUARY 28, 1980. NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

REVIEWED BY OMPA LOEB: T. C. CINTULA **DATE:** 5·9·80

REVIEWED BY NRR DOL: **DATE:** V.12.1980

FORT ST. VRAIN

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-267
2. Utility Contact: J. W. GAHM (303) 785-2253
3. Licensed Thermal Power (MWt): 842
4. Nameplate Rating (Gross MWe): 403 x 0.85 = 343
5. Design Electrical Rating (Net MWe): 330
6. Maximum Dependable Capacity (Gross MWe): 342
7. Maximum Dependable Capacity (Net MWe): 330
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): 231
10. Reasons for Restrictions, if Any:
NRC RESTRICT. 70% PEND. RESOL. OF TEMP. FLUCTUATIONS

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	2,184.0	6,601.0
12. Hours Reactor Was Critical	676.1	863.9	3,396.7
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	358.0	358.0	1,340.2
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	157,856	161,148	639,103
17. Gross Electrical Energy (MWH)	48,427	48,427	189,223
18. Net Electrical Energy (MWH)	43,047	43,047	166,631
19. Unit Service Factor	48.1	16.4	20.3
20. Unit Availability Factor	48.1	16.4	20.3
21. Unit Capacity Factor (MDC Net)	17.5	6.0	7.6
22. Unit Capacity Factor (DER Net)	17.5	6.0	7.6
23. Unit Forced Outage Rate	44.1	44.1	61.2

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type¹	Duration (Hours)	Reason²	Method of Shutting Down Reactor³	Licensee Event Report No.	System Code⁴	Component Code⁵	Cause & Corrective Action to Prevent Recurrence
79-14	10/26/79	S	98.8	B	4				SCHEDULED SHUTDOWN CONTINUED FOR REPLACEMENT OF RUPTURED STATIC SEAL. REFER TO LER 80-001/01-T-0. GENERATOR PUT ON LINE 3580
80-01	03/06/80	S	4.2	B	2				TURBINE OVER-SPEED TRIP TEST.
80-02	03/11/80	F	123.8	A	2				CIRCULATING WATER SYSTEM 41 SHUTDOWN DUE TO TOWER PROBLEMS
80-03	03/21/80	F	56.8	A	3				REACTOR SCRAMMED DUE TO INSTRUMENT PROBLEMS
80-04	03/24/80	F	49.2	A	2				LOSS OF CONDENSER VACUUM.
80-05	03/28/80	F	53.2	A	1				1A CIRCULATOR TRIP AND SYSTEM UPSET.

SUMMARY

UNIT RETURNED TO 70% POWER LEVEL WITH FIVE SHUTDOWNS THIS MONTH.

LEGEND

¹Type:
F--Forced
S--Scheduled

²Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

³Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

⁴Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

⁵Exhibit H:
Same source.

GINNA

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	New York/Wayne 15 miles NE of Rochester
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	1520
5.	Design Electrical Rating (MWE Net)	470
6.	Date of Initial Criticality	November 8, 1969
7.	Date of Commercial Operation	July 1970
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Lake Ontario

Utility & Contractor Information

10.	Licensee	Rochester Gas & Electric Corporation
11.	Corporate Address	89 East Avenue Rochester, New York
12.	Corporate Contact	Leon D. White, Jr., Vice President Electric and Steam Production
13.	Architect/Engineer	Gilbert
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Senior Inspector	R. Keimig
19.	IE Principal Inspector	R. S. Markowski
20.	Licensing Project Manager	S. J. Nowicki
21.	Docket Number	50-244
22.	License Number/Date of Issue	DPR-18, 9/19/69
23.	Public Document Room	Rochester Public Library Business and Social Science Division 115 South Avenue Rochester, New York 14604

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS:

- Shutdown for refueling.

LAST IE SITE INSPECTION DATE: 3/6/80

INSPECTION REPORT NO: 50-244/80-04

SUMMARY

INSPECTION SUMMARY

- Reports 80-02, 80-03 and 80-04 not completed.

REPORTS RECEIVED FROM LICENSEE

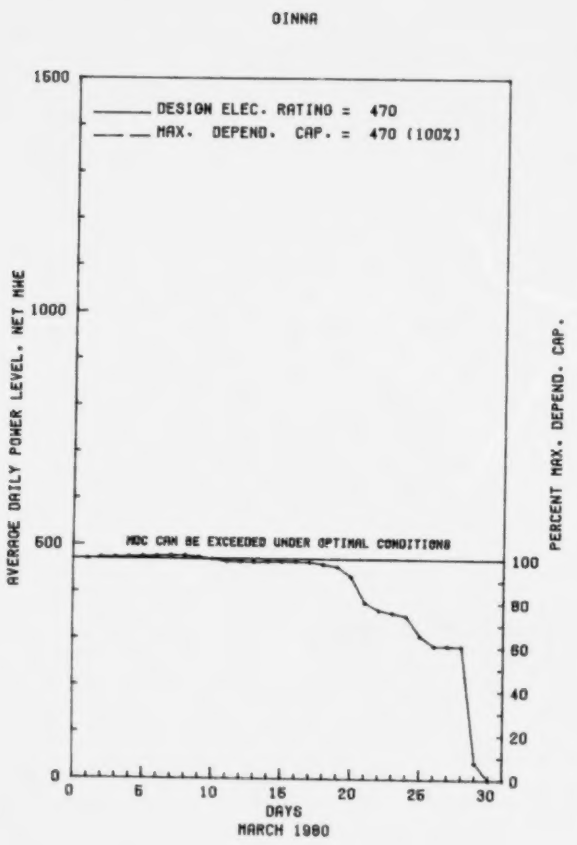
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
NONE				

REVIEWED BY OMPA LOEB: T. C. Cintula **DATE:** 5. 9. 80

REVIEWED BY NRR DOR: **DATE:** 5-13-80

GINNA

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-244
2. Utility Contact: A.E. MCNAMARA (315) 524-4446
3. Licensed Thermal Power (MWt): 1520
4. Nameplate Rating (Gross MWe): 608 x 0.85 = 517
5. Design Electrical Rating (Net MWe): 470
6. Maximum Dependable Capacity (Gross MWe): 490
7. Maximum Dependable Capacity (Net MWe): 470

CAPACITY FACTORS CALCULATED WITH A WEIGHTED AVERAGE

8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	2,184.0	90,696.0
12. Hours Reactor Was Critical	674.5	2,114.5	69,794.3
13. Reactor Reserve Shutdown Hours	.0	.0	1,621.8
14. Hours Generator On-Line	674.5	2,114.5	68,071.3
15. Unit Reserve Shutdown Hours	.0	.0	8.5
16. Gross Thermal Energy (MWH)	935,736	3,118,656	91,394,458
17. Gross Electrical Energy (MWH)	302,276	1,021,206	29,706,997
18. Net Electrical Energy (MWH)	286,350	970,811	28,132,359
19. Unit Service Factor	90.7	96.8	75.1
20. Unit Availability Factor	90.7	96.8	75.1
21. Unit Capacity Factor (MDC Net)	81.9	94.6	68.4
22. Unit Capacity Factor (DER Net)	81.9	94.6	68.4
23. Unit Forced Outage Rate	.0	.0	9.6

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 05/10/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
1	03/29/80	S	69.5	C	1				ANNUAL REFUELING AND MAINTENANCE. SHUTDOWN HOURS IN MONTH OF MARCH ONLY.

SUMMARY

OPERATION WAS ROUTINE UNTIL UNIT SHUTDOWN ON 03/29/80 FOR A REFUELING AND MAINTENANCE OUTAGE. LOAD REDUCTIONS OCCURRED ON 03/10/80 AND 03/21/80 DUE TO PROBLEMS WITH THE 5B FEEDWATER HEATER, AND ON 03/17/80 WHEN END-OF-CYCLE COASTDOWN BEGAN

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

HADDAM NECK

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Connecticut/Middlesex 13 miles E of Meriden
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	1825
5.	Design Electrical Rating (MWE Net)	575
6.	Date of Initial Criticality	July 24, 1967
7.	Date of Commercial Operation	January 1, 1968
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Connecticut River

Utility & Contractor Information

10.	Licensee	Connecticut Yankee Atomic Power Co.
11.	Corporate Address	P. O. Box 270 Hartford, Connecticut
12.	Corporate Contact	D. Switzer, President
13.	Architect/Engineer	Stone & Webster
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Senior Inspector	R. Keimig
19.	IE Principal Inspector	W. J. Lazarus
20.	Licensing Project Manager	R. Caruso
21.	Docket Number	50-213
22.	License Number/Date of Issue	DPR-61, 12/27/74
23.	Public Document Room	Russell Library 119 Broad Street Middletown, Connecticut 06457

INSPECTION STATUS

ENFORCEMENT STATUS

ETS 2.4.3.1(1) LIMITS THE RELEASE RATE OF NOBLE GASES FROM THE SITE BASED UPON THE BETA & GAMMA ANALYSIS OF THE ISOTOPE MIX BEING RELEASED. CONTRARY TO THE ABOVE REQUIREMENT, ON 12/16/79, A NOBLE GAS MIXTURE WAS RELEASED WHICH EXCEEDED THE TS 2.4.3.1(1) LIMIT BY A FACTOR OF 5.2 FOR A PERIOD OF 10 MINUTES.
(7921 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

FACILITY ITEMS (PLANS AND PROCEDURES):

+ An 8 week refueling outage is scheduled to begin May 3.

PLANT STATUS:

+ No significant information to report.

+ LAST IE SITE INSPECTION DATE: 4/9-11/80

+ INSPECTION REPORT NO: 50-213/80-06

SUMMARY

INSPECTION SUMMARY

+ 50-213/80-03 - Feb 20-22: Routine, unannounced inspection by two region-based inspectors (24 hours) of plant operations and monthly operating reports received in the RI office. Noncompliances: None.

REPORTS RECEIVED FROM LICENSEE

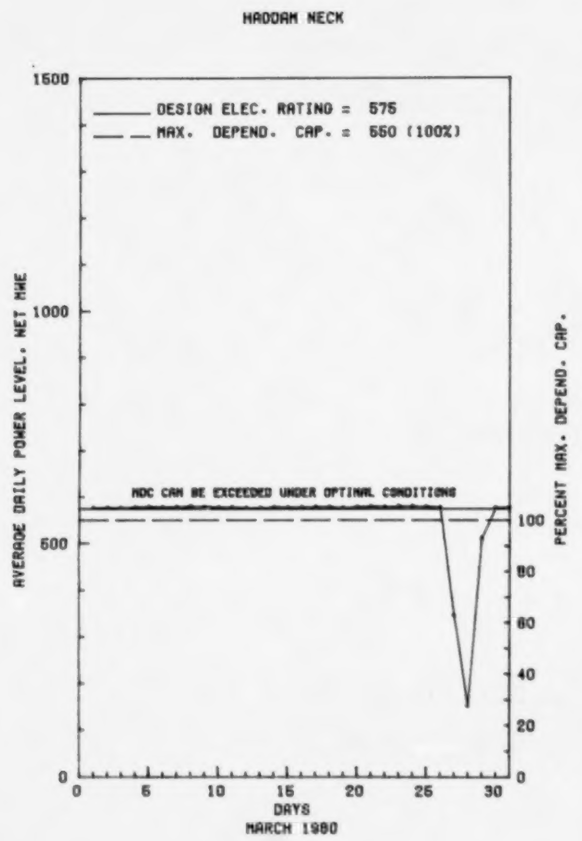
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
80-001E	02/23/80	03/03/80	Environmental Deviation	Chlorine discharge limit to river exceeded
80-003/01T	01/29/80	02/12/80	24 Hours	Description of sequence of events which could cause emergency diesel overloading
80-004/03L	02/04/80	02/12/80	30 Day LER	Spurious opening of PORV
80-005/03L	02/07/80	02/29/80	30 Day LER	Leak on boron waste storage tank thermo syphon heater. Leak repaired with no release or personnel exposure

REVIEWED BY OMPA LOEB: T.C. Cintula DATE: 5-9-80

REVIEWED BY NRR DOR: R Caruso DATE: 5/13/80

HADDAM NECK

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-213
2. Utility Contact: C. Kokoszka (203) 267-2556
3. Licensed Thermal Power (MWt): 1825
4. Nameplate Rating (Gross MWe): 667 x 0.9 = 600
5. Design Electrical Rating (Net MWe): 575
6. Maximum Dependable Capacity (Gross MWe): 577
7. Maximum Dependable Capacity (Net MWe): 550
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

CAPACITY FACTORS CALCULATED WITH A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	2,184.0	107,376.0
12. Hours Reactor Was Critical	732.6	2,172.6	93,369.3
13. Reactor Reserve Shutdown Hours	11.4	11.4	1,164.0
14. Hours Generator On-Line	730.3	2,170.3	89,110.5
15. Unit Reserve Shutdown Hours	.0	.0	365.9
16. Gross Thermal Energy (MWH)	1,301,936	3,858,973	154,000,122
17. Gross Electrical Energy (MWH)	431,516	1,278,967	50,633,413
18. Net Electrical Energy (MWH)	411,806	1,220,391	48,175,154
19. Unit Service Factor	98.2	99.4	83.0
20. Unit Availability Factor	98.2	99.4	83.3
21. Unit Capacity Factor (MDC Net)	100.6	101.6	83.0
22. Unit Capacity Factor (DER Net)	96.3	97.2	77.2
23. Unit Forced Outage Rate	1.8	.6	7.1

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUELING, MAY 1980, APPROXIMATELY 8 TO 12 WEEKS
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
12	03/27/80	F	13.7	H	3		ZZ	ZZZZZZ	REACTOR AND TURBINE TRIP DUE TO HIGH CONTAINMENT PRESSURE/CORE COOLING ACTUATION SIGNAL.

SUMMARY

OPERATION WAS AT NEAR FULL POWER DURING THE REPORT PERIOD, EXCEPT FOR A REACTOR/TURBINE TRIP ON 03/27/80 DURING WIRING MODIFICATIONS PER NUREG-0578.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source

NRC POWER RESTRICTIONS: NONE

HATCH 1

FACILITY DATA

Facility Description

1. Location (state/county) (nearest population center)	Georgia/Appling 11 miles N of Baxley
2. Electric Reliability Council	Southeastern Electric Reliability Council
3. Type of Reactor	BWR
4. Power Level (MWT)	2436
5. Design Electrical Rating (MWE Net)	777
6. Date of Initial Criticality	September 12, 1974
7. Date of Commercial Operation	December 31, 1975
8. Condenser Cooling Method	Once Through With Forced Draft Cooling Towers
9. Condenser Cooling Water Source	Altamaha River

Utility & Contractor Information

10. Licensee	Georgia Power Company
11. Corporate Address	270 Peachtree Street, NE Atlanta, Georgia 30303
12. Corporate Contact	J. H. Miller Executive Vice President
13. Architect/Engineer	Southern Services, Inc.
14. Nuclear Steam Supply System	GE
15. Constructor	Georgia Power Company

Regulatory Information

16. IE Region responsible	II
17. IE Branch Chief	R. C. Lewis
18. IE Senior Inspector	H. C. Dance
19. IE Resident Inspector	R. F. Rogers (912) 367-9881
20. Licensing Project Manager	D. Verrelli
21. Docket Number	50-321
22. License Number/Date of Issue	DPR-57, 8/6/74
23. Public Document Room	Appling County Public Library Parker Street Baxley, Georgia 31513

INSPECTION STATUS

ENFORCEMENT STATUS

AS REQUIRED BY TECHNICAL SPECIFICATION TABLES 4.1-1 (ITEM 3) AND 4.2-7 (ITEM 2), THE INTERMEDIATE RANGE MONITORS (IRM) ARE TO BE FUNCTIONALLY TESTED IN ORDER TO ASSURE OPERABILITY. AS DESCRIBED BY TECHNICAL SPECIFICATION, SECTION 1.0, DEFINITIONS, A CHANNEL FUNCTIONAL TEST INJECTS A SIMULATED SIGNAL INTO THE CHANNEL SENSOR TO VERIFY CHANNEL OPERABILITY INCLUDING ALARM AND/OR TRIP FUNCTIONS AND CHANNEL FAILURE TRIPS. CONTRARY TO THE ABOVE, IRM CHANNELS ARE NOT BEING FUNCTIONALLY TESTED TO ASSURE OPERABILITY OF ALL ALARMS, TRIPS AND INDICATIONS FROM THE SENSOR TO PANEL H11-P603 IN THE CONTROL ROOM.
(8005 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

FULL POWER.

LAST IE SITE INSPECTION DATE: MARCH 26-28, 1980

INSPECTION REPORT NO: 50-321/80-16

SUMMARY

INSPECTION SUMMARY

INSPECTION JANUARY 28-31 (80-05): THIS ROUTINE UNANNOUNCED INSPECTION INVOLVED 26 INSPECTOR-HOURS ON SITE. THE AREAS INSPECTED WERE MEASUREMENTS OF CORE PERFORMANCE ON UNITS 1 AND 2, RESPONSE TO LERS ON BOTH UNITS, INITIAL STARTUP TESTS FOR UNIT 2 AND FOLLOWUP ON AN EARLIER UNRESOLVED ITEM. OF THE FOUR AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN TWO AREAS. ONE APPARENT ITEM OF NONCOMPLIANCE WAS IDENTIFIED IN ONE AREA (INFRACTION - FAILURE TO PERFORM CHANNEL FUNCTIONAL TEST). ONE APPARENT DEVIATION WAS FOUND IN ONE AREA (FAILURE TO CONFORM TO A COMMITMENT FOR CORRECTIVE ACTION MADE IN AN LER).

INSPECTION FEBRUARY 19-21 (80-96): THIS SPECIAL, ANNOUNCED INSPECTION INVOLVED 23 INSPECTOR-HOURS ON SITE IN THE AREAS OF SMALL BREAK LOSS OF COOLANT ACCIDENT PROCEDURES AND TRAINING. OF THE TWO AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN ONE AREA. ONE DEVIATION WAS FOUND IN ONE AREA (DEVIATION - FAILURE TO ENSURE THE STATIONS LICENSED OPERATORS HAVE COMMITTED TO MEMORY THE IMMEDIATE ACTIONS OF THE FACILITY'S EMERGENCY PROCEDURES).

INSPECTION FEBRUARY 21-22 (80-08): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 5 INSPECTOR-HOURS ON SITE IN THE AREAS OF FOLLOWUP ON ITEMS OR NONCOMPLIANCE RELATIVE TO 10 CFR PART 21. THE PREVIOUSLY IDENTIFIED ITEMS OF NONCOMPLIANCE IDENTIFIED IN IE REPORTS 50-321/79-33 AND 50-366/79-37 REMAIN OPEN.

INSPECTION JANUARY 5 - FEBRUARY 2 (80-10): THIS INSPECTION INVOLVED 45 INSPECTOR-HOURS ON SITE OF TECHNICAL SPECIFICATION COMPLIANCE, REPORTABLE OCCURRENCES, HOUSEKEEPING, OPERATOR PERFORMANCE, OVERALL PLANT OPERATIONS, QUALITY ASSURANCE PRACTICES, STATION AND CORPORATE MANAGEMENT PRACTICES, CORRECTIVE AND PREVENTATIVE MAINTENANCE ACTIVITIES, SITE SECURITY PROCEDURES, RADIATION CONTROL ACTIVITIES, SURVEILLANCE ACTIVITIES, AND FOLLOWUP OF PREVIOUS INSPECTION FINDINGS. OF THE TWELVE AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

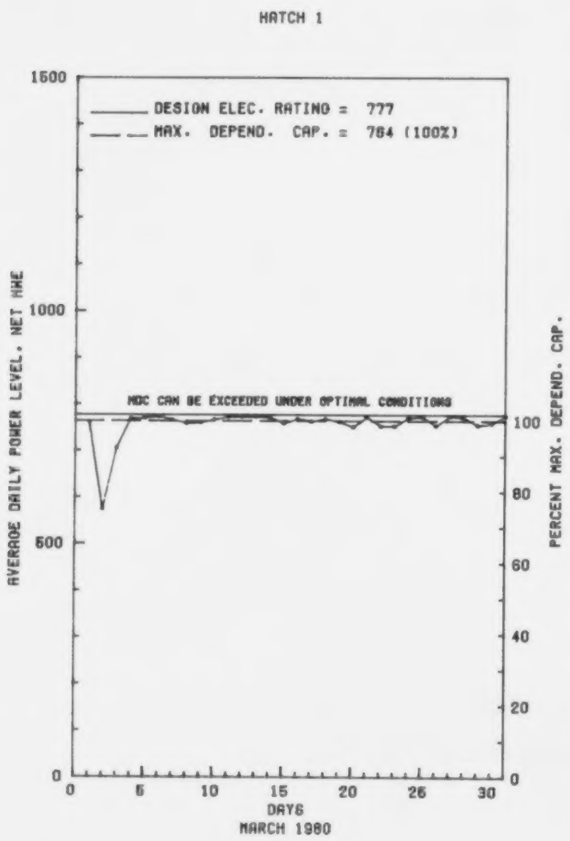
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
80-014/ 03L-0	01/30/80	02/18/80	30 DAY LER	REACTOR BUILDING VENT STACK RADIATION MONITOR INOPERABLE.
80-015/ 03L-0	02/07/80	02/26/80	30 DAY LER	DRYWELL TEMPERATURE RECORDER MALFUNCTION.
80-018/ 03L-0	02/07/80	02/25/80	30 DAY LER	CONTROL ROD DRIVE ACCUMULATOR INOPERABLE.
80-019/ 03L-0	02/13/80	03/04/80	30 DAY LER	REACTOR CORE ISOLATION COOLING TURBINE EXHAUST PRESS SWITCHES SETPOINTS FOUND OUTSIDE TECHNICAL SPECIFICATION LIMITS.
80-023/ 03L-0	02/24/80	03/10/80	30 DAY LER	HIGH PRESSURE COOLANT INJECTION SYSTEM DID NOT BECOME OPERATIONAL AT 113 PSIG DURING START UP AS REQUIRED BY TECHNICAL SPECIFICATIONS.

REVIEWED BY OMPA LOEB: **DATE:** 5-12-80

REVIEWED BY NRR DOR: **DATE:** 5-13-80

HATCH 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-321
2. Utility Contact: J. Roden (912) 367-7781
3. Licensed Thermal Power (MWt): 2436
4. Nameplate Rating (Gross MWe): 1000 x 0.85 = 850
5. Design Electrical Rating (Net MWe): 777
6. Maximum Dependable Capacity (Gross MWe): 797
7. Maximum Dependable Capacity (Net MWe): 764
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	2,184.0	37,248.0
12. Hours Reactor Was Critical	744.0	1,964.8	27,762.8
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	744.0	1,889.8	26,158.2
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,760,740	4,306,210	54,659,245
17. Gross Electrical Energy (MWH)	587,040	1,433,050	17,703,990
18. Net Electrical Energy (MWH)	563,323	1,370,842	16,832,775
19. Unit Service Factor	100.0	86.5	70.2
20. Unit Availability Factor	100.0	86.5	70.2
21. Unit Capacity Factor (MDC Net)	99.1	82.2	59.2
22. Unit Capacity Factor (DER Net)	97.4	80.8	58.2
23. Unit Forced Outage Rate	.0	13.5	15.9

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-09	03/02/80	S	.0	H	5		ZZ		CONROD REDUCTION FOR ROD PATTERN ADJUSTMENT.

SUMMARY

OPERATION WAS AT NEAR FULL POWER DURING THE REPORT PERIOD, EXCEPT FOR A LOAD REDUCTION ON 03/02/80 FOR CONTROL ROD ADJUSTMENT.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

HATCH 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Georgia/Appling 11 miles N of Baxley
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2436
5.	Design Electrical Rating (MWE Net)	784
6.	Date of Initial Criticality	July 4, 1978
7.	Date of Commercial Operation	September 5, 1979
8.	Condenser Cooling Method	Cooling Towers
9.	Condenser Cooling Water Source	Altamaha River

Utility & Contractor Information

10.	Licensee	Georgia Power Company
11.	Corporate Address	270 Peach Street, NE Atlanta, Georgia 30303
12.	Corporate Contact	J. H. Miller Executive Vice President
13.	Architect/Engineer	Southern Services, Inc./Bechtel
14.	Nuclear Steam Supply System	GE
15.	Constructor	Georgia Power Company

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. Lewis
18.	IE Senior Inspector	H. C. Dance
19.	IE Resident Inspector	R. F. Rogers (912) 367-9881
20.	Licensing Project Manager	D. Verrelli
21.	Docket Number	50-366
22.	License Number/Date of Issue	NPF-5/June 13, 1978
23.	Public Document Room	Appling County Public Library Parker Street Baxley, Georgia 31513

INSPECTION STATUS

ENFORCEMENT STATUS

AS REQUIRED BY TECHNICAL SPECIFICATION TABLES 4.1-1 (ITEM 3) AND 4.2-7 (ITEM 2), THE INTERMEDIATE RANGE MONITORS (IRM) ARE TO BE FUNCTIONALLY TESTED IN ORDER TO ASSURE OPERABILITY. AS DESCRIBED BY TECHNICAL SPECIFICATION, SECTION 1.0, DEFINITIONS, A CHANNEL FUNCTIONAL TEST INJECTS A SIMULATED SIGNAL INTO THE CHANNEL SENSOR TO VERIFY CHANNEL OPERABILITY INCLUDING ALARM AND/OR TRIP FUNCTIONS AND CHANNEL FAILURE TRIPS. CONTRARY TO THE ABOVE, IRM CHANNELS ARE NOT BEING FUNCTIONALLY TESTED TO ASSURE OPERABILITY OF ALL ALARMS, TRIPS AND INDICATIONS FROM THE SENSOR TO PANEL H11-P603 IN THE CONTROL ROOM.
(8005 INFRACTION)

ON THREE SEPARATE OCCASIONS DURING 1979, AS DOCUMENTED IN LERS 79-087, 79-107 AND 79-121, AVERAGE PLANAR LINEAR HEAT GENERATION RATES WERE EXCEEDED SHORTLY AFTER REACTOR POWER WAS RETURNED TO 100% FOLLOWING A MINOR LOAD REDUCTION TO PERFORM WEEKLY SURVEILLANCE TESTS OR TO CORRECT EQUIPMENT PROBLEMS. DUE TO THE REPETITIVE NATURE AND CIRCUMSTANCES SURROUNDING THESE EVENTS, THE LICENSEE COMMITTED TO THE NRC IN LER 79-107 DATED OCTOBER 22, 1979 THAT THE FACILITY WOULD REVISE ITS PROCEDURES WITHIN THIRTY DAYS TO INSTRUCT OPERATIONS PERSONNEL ON THE CORRECT ACTIONS TO TAKE ON INCREASING POWER FOLLOWING A LOAD REDUCTION OF SHORT DURATION. CONTRARY TO THE ABOVE, ON JANUARY 31, 1980, THE INSPECTOR DISCOVERED THAT THE COMMITTED CHANGES HAD NOT BEEN PERFORMED.
(8005 DEVIATION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

+ PLANT IS SHUTDOWN FOR MAINTENANCE UNTIL APRIL 19, 1980.

+ LAST IE SITE INSPECTION DATE: MARCH 26-28, 1980

+ INSPECTION REPORT NO: 50-366/80-16

SUMMARY

INSPECTION SUMMARY

+ INSPECTION JANUARY 28-31 (80-05): THIS ROUTINE UNANNOUNCED INSPECTION INVOLVED 26 INSPECTOR-HOURS ON SITE. THE AREAS INSPECTED WERE MEASUREMENTS OF CORE PERFORMANCE ON UNITS 1 AND 2, RESPONSE TO LERS ON BOTH UNITS, INITIAL STARTUP TESTS FOR UNIT 2 AND FOLLOWUP ON AN EARLIER UNRESOLVED ITEM. OF THE FOUR AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN TWO AREAS. ONE APPARENT ITEM OF NONCOMPLIANCE WAS IDENTIFIED IN ONE AREA (INFRACTION - FAILURE TO PERFORM CHANNEL FUNCTIONAL TEST). ONE APPARENT DEVIATION WAS FOUND IN ONE AREA (FAILURE TO CONFORM TO A COMMITMENT FOR CORRECTIVE ACTION MADE IN AN LER).

INSPECTION FEBRUARY 19-21 (80-06): THIS SPECIAL, ANNOUNCED INSPECTION INVOLVED 23 INSPECTOR-HOURS ON SITE IN THE AREAS OF SMALL BREAK LOSS OF COOLANT ACCIDENT PROCEDURES AND TRAINING. OF THE TWO AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN ONE AREA. ONE DEVIATION WAS FOUND IN ONE AREA (DEVIATION - FAILURE TO ENSURE THE STATIONS LICENSED OPERATORS HAVE COMITTED TO MEMORY THE IMMEDIATE ACTIONS OF THE FACILITY'S EMERGENCY PROCEDURES).

INSPECTION FEBRUARY 21-22 (80-08): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 5 INSPECTOR-HOURS ON SITE IN THE AREAS OF FOLLOWUP ON ITEMS OR NONCOMPLIANCE RELATIVE TO 10 CFR PART 21. THE PREVIOUSLY IDENTIFIED ITEMS OF NONCOMPLIANCE IDENTIFIED IN IE REPORTS 50-321/79-33 AND 50-366/79-37 REMAIN OPEN.

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
80-012/ 03L-0	02/14/80	02/26/80	30 DAY LER	RESIDUAL HEAT REMOVAL AND PLANT SERVICE WATER MINIMUM FLOW LINES INADEQUATELY SUPPORTED.
80-013/ 03L-0	02/03/80	02/20/80	30 DAY LER	REACTOR CORE ISOLATION COOLING CONTROLS INOPERATIVE AT REMOTE SHUTDOWN BOARD.

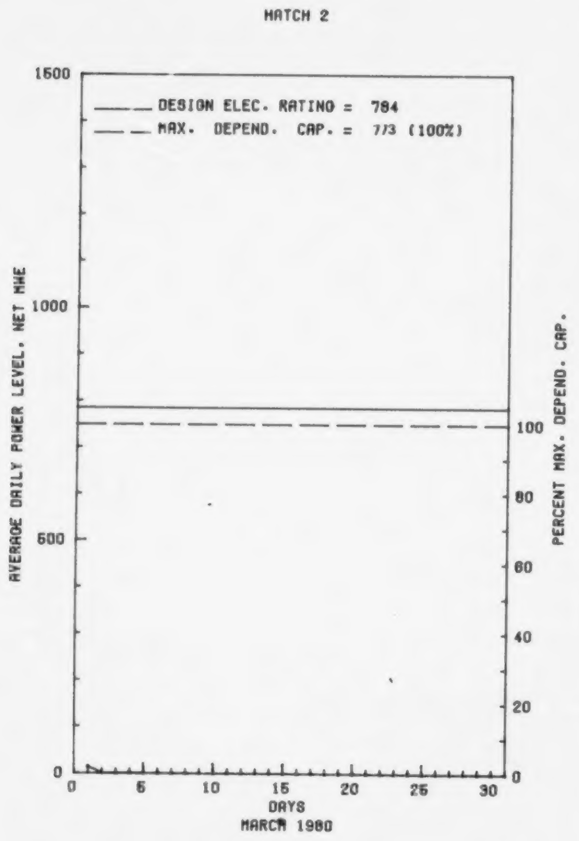
INSPECTION JANUARY 5 - FEBRUARY 2 (80-10): THIS INSPECTION INVOLVED 45 INSPECTOR-HOURS ON SITE OF TECHNICAL SPECIFICATION COMPLIANCE, REPORTABLE OCCURRENCES, HOUSEKEEPING, OPERATOR PERFORMANCE, OVERALL PLANT OPERATIONS, QUALITY ASSURANCE PRACTICES, STATION AND CORPORATE MANAGEMENT PRACTICES, CORRECTIVE AND PREVENTATIVE MAINTENANCE ACTIVITIES, SITE SECURITY PROCEDURES, RADIATION CONTROL ACTIVITIES, SURVEILLANCE ACTIVITIES, AND FOLLOWUP OF PREVIOUS INSPECTION FINDINGS. OF THE TWELVE AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

REVIEWED BY OMPA LOEB: John J. Pezzio **DATE:** 5-12-80

REVIEWED BY NRR DOR: **DATE:** 5-13-80

HATCH 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-366
2. Utility Contact: J. Rodan (912) 367-7781
3. Licensed Thermal Power (MWt): 2436
4. Nameplate Rating (Gross MWe): 1000 x .85 = 850
5. Design Electrical Rating (Net MWe): 784
6. Maximum Dependable Capacity (Gross MWe): 806
7. Maximum Dependable Capacity (Net MWe): 773
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
9. Power Level To Which Restricted, If Any (Ne. MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	2,184.0	5,017.0
12. Hours Reactor Was Critical	12.6	1,387.0	4,050.5
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	3.0	1,344.5	3,759.2
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,920	3,062,580	8,556,642
17. Gross Electrical Energy (MWH)	630	1,031,980	2,867,940
18. Net Electrical Energy (MWH)	-3,211	985,667	2,742,804
19. Unit Service Factor	.4	61.6	74.9
20. Unit Availability Factor	.4	61.6	74.9
21. Unit Capacity Factor (MDC Net)	.0	58.4	70.8
22. Unit Capacity Factor (DER Net)	.0	57.6	69.7
23. Unit Forced Outage Rate	.0	1.2	5.3

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 5/14/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-06	03/01/80	S	741.0	H	2		SH	ZZZZZZ	VENT HEADER DEFLECTOR AND 18-MONTH SURVEILLANCE OUTAGE.

SUMMARY

THE UNIT WAS SHUTDOWN ON 03/01/80 FOR A PERIODIC SURVEILLANCE OUTAGE AND VENT HEADER DEFLECTOR MAINTENANCE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

HUMBOLDT BAY

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	California/Humboldt 4 miles S of Eureka
2.	Electric Reliability Council	Western Systems Coordinating Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	220
5.	Design Electrical Rating (MWE Net)	65
6.	Date of Initial Criticality	February 16, 1963
7.	Date of Commercial Operation	August 1963
8.	Condenser Cooling Method	Circulating sea water
9.	Condenser Cooling Water Source	Pacific Ocean (Humboldt Bay)

Utility & Contractor Information

10.	Licensee	Pacific Gas & Electric Co.
11.	Corporate Address	77 Beale Street San Francisco, California 94106
12.	Corporate Contact	P. A. Crane, Jr. Asst. General Counsel
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	GE
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	V
17.	IE Branch Chief	J. L. Crews
18.	IE Senior Inspector	D. M. Sternberg
19.	IE Principal Inspector	J. D. Carlson
20.	Licensing Project Manager	V. L. Rooney
21.	Docket Number	50-133
22.	License Number/Date of Issue	DPR-7, 8/28/62
23.	Public Document Room	Humboldt County Library 636 F Street Eureka, California 95501

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST (CITE RO REPORT NUMBER, IF APP.)

SYSTEMS AND COMPONENT PROBLEMS:

None.

FACILITY ITEMS (PLANS AND PROCEDURES):

+ The plant has been shut down since July 1976.

Several items remaining to be completed during the outage have been deferred pending resolution of the seismic issues at the plant site. A date for return to operations has not been established.

MANAGERIAL ITEMS:

None

PLANT STATUS:

Plant is shutdown.

LAST IE SITE INSPECTION DATE: 03/24-28/80+

INSPECTION REPORT NO: 50-133/80-01+

LICENSE STATUS

Order issued 5/21/76 requiring Utility to prove seismic qualification before resuming operation of unit after outage of 7/2/76.

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
80-01/ 03L-0	02-10-80	03-11-80	30 Day LER	Setpoint drift caused fire pumps to start early during routine test.

SUMMARY

INSPECTION SUMMARY

+ Inspection on March 24-28, 1980 (Report No. 50-133/80-01)
Routine, unannounced inspection by Regional based inspectors of the licensee's response to IE Bulletin 79-19, "Packaging of Low-Level Radioactive waste for Transport and Burial;" IE Circular Nos. 79-09, 79-15, 79-21; and the licensee's laboratory counting room equipment quality control program. Areas inspected included: tours of the facility; review of regulatory requirements; burial site requirements; procedures; training; audits; inspection of packages and containers; and review of records associated with shipments of radioactive materials. In addition, the licensee submitted a sample of liquid effluent for independent analysis by the NRC and performed isotopic identification and quantification of samples supplied by the NRC. The inspection involved 38 inspector hours on site by two NRC Regional based inspectors

Results: Of the ten areas inspected, no items of noncompliance were identified in nine areas; three apparent items of noncompliance were identified in one area (Infraction - Failure to post a high radiation area, 10 CFR 20.203(c); Infraction - Failure to control access to high radiation areas, 10 CFR 20.203(c)(2)(iii); Deficiency - Failure to label a container of licensed material, 10 CFR 20.203(f).

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** ~~8~~5-12-80

REVIEWED BY NRR DOR: **DATE:** 5/12/80

HUMBOLDT BAY

AVERAGE DAILY POWER LEVEL (MWe)

OPERATING STATUS

1. Docket Number: 50-133
2. Utility Data Prepared By: B. C. Getty (707) 443-2787
3. Licensed Thermal Power (MWt): 220
4. Nameplate Rating (Gross MWe): 77 X 0.85 = 65
5. Design Electrical Rating (Net MWe): 65
6. Maximum Dependable Capacity (Gross MWe): 0
7. Maximum Dependable Capacity (Net MWe): 0
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons

None

9. Power Level To Which Restricted, If Any (Net MWe): None
10. Reasons For Restrictions, If Any:

This ... ive

11. Hours In Reporting Period
12. Number Of Hours Reactor Was Cri
13. Reactor Reserve Shutdow
14. Hours Generato
15. Unit R
16. ... rt)
17. ... erated (MWH)
18. Net ... gy Generated (MWH)
19. Unit S... ce Factor
20. Unit Availability Factor
21. Unit Capacity Factor (Using MDC Net) 1/
22. Unit Capacity Factor (Using DER Net) 1/
23. Unit Forced Outage Rate
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

None

25. If Shut Down At End Of Report Period, Estimated Date of Startup: UNKNOWN

NO DECISION ON FUTURE OPERATION
SHUTDOWN INDEFINITELY

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
SEE SUMMARY BELOW									

SUMMARY

THE UNIT WAS SHUTDOWN ON 7/2/76 FOR SEISMIC MODIFICATIONS. THE UNIT REMAINS IN THE COLD SHUTDOWN CONDITION.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

INDIAN POINT 1

FACILITY DATA

Facility Description

1. Location (state/countv) (nearest population center)	New York/Westchester 25 miles N of New York City
2. Electric Reliability Council	Northeast Power Coordinating Council
3. Type of Reactor	PWR/fossil superheat
4. Power Level (MWT)	615
5. Design Electrical Rating (MWE Net)	265
6. Date of Initial Criticality	August 2, 1962
7. Date of Commercial Operation	October 1962
8. Condenser Cooling Method	Once Through
9. Condenser Cooling Water Source	Hudson River

Utility & Contractor Information

10. Licensee	Consolidated Edison Company
11. Corporate Address	of New York, Inc. 4 Irving Place New York, New York
12. Corporate Contact	W. Cahill, Vice President
13. Architect/Engineer	Utility
14. Nuclear Steam Supply System	B&W
15. Constructor	Utility

Regulatory Information

16. IE Region responsible	I
17. IE Branch Chief	E. J. Brunner
18. IE Senior Inspector	R. Keimig
19. IE Resident Inspector	T. Rebelowski (914-739-9360) FTS 656-9220
20. Licensing Project Manager	L. Olshan x27144
21. Docket Number	50-03
22. License Number/Date of Issue	DPR-5, 3/26/62
23. Public Document Room	White Plains Public Library 100 Martine Avenue White Plains, New York 10601

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

FACILITY ITEMS (PLANS AND PROCEDURES):

+ Licensee has submitted his intentions to decommission plant at a unspecified date based on operating history of Unit 2.

PLANT STATUS.

Defueled.

LAST IE SITE INSPECTION DATE: 7/24-27/79

INSPECTION REPORT NO: 50-03/79-02

3. SUMMARY

INSPECTION SUMMARY

None.

LICENSE STATUS

The unit produced electricity to the expiration date (October 31, 1974) of delay granted by the AEC for installation of an ECCS system. The ECCS system was not subsequently installed and the unit was defueled on January 30, 1976.

On February 11, 1980, the NRC requested the utility to show cause as to why the operating authority of the license should not be revoked and why the licensee should not submit a plan within 120 days of this order to decommission the unit.

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
NONE				

REVIEWED BY OMPA LOEB: T. C. CINTULA DATE: 5-9-80

REVIEWED BY NRR DOR: DATE: 5-12-80

INDIAN POINT 1

AVERAGE DAILY POWER LEVEL (MWe)

OPERATING STATUS

1. Docket Number: 50-003
2. Utility Data Prepared By: L. J. Kawula (914)-694-6000 x209
3. Licensed Thermal Power (MWt): 615
4. Nameplate Rating (Gross MWe): 275
5. Design Electrical Rating (Net MWe): 265
6. Maximum Dependable Capacity (Gross MWe): 0
7. Maximum Dependable Capacity (Net MWe): 0
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons
None
9. Power Level To Which Restricted, If Any (Net MWe) None
10. Reasons For Restrictions, If Any

This Month Yr. to Date

11. Hours In Reporting Period
12. Number Of Hours Reactor Was Critical
13. Reactor Reserve Shutdown Hours
14. Hours Generator On-Line
15. Unit Reserve Shutdown Ho
16. Gross Thermal E
17. Gros
20. Unit A
21. Unit Capacity Factor (U Net) 1/
22. Unit Capacity Factor (Using DER Net) 1/
23. Unit Forced Outage Rate
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
None
25. If Shut Down At End Of Report Period, Estimated Date of Startup: N/A

NO DECISION ON FUTURE OPERATION
SHUTDOWN INDEFINITELY
ALL FUEL UNLOADED.

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
SEE SUMMARY BELOW									

SUMMARY

UNIT SHUTDOWN ON OCTOBER 31, 1974, ALL FUEL HAS BEEN UNLOADED; NO DECISION ON DECOMMISSIONING DATE.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

INDIAN POINT 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	New York/Westchester 25 miles N of New York City
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2758
5.	Design Electrical Rating (MWE Net)	873
6.	Date of Initial Criticality	May 22, 1973
7.	Date of Commercial Operation	August 1973
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Hudson River

Utility & Contractor Information

10.	Licensee	Consolidated Edison Company of New York, Inc.
11.	Corporate Address	4 Irving Place New York, New York
12.	Corporate Contact	W. J. Cahill, Jr., Vice President
13.	Architect/Engineer	UE&C
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	J. A. Jones/UE&C

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Senior Inspector	R. Keimig
19.	IE Resident Inspector	T. Rebelowski (914-739-9360) FTS 656-9220
20.	Licensing Project Manager	L. Olshan (X27144)
21.	Docket Number	50-247
22.	License Number/Date of Issue	DPR-26, 10/19/71
23.	Public Document Room	White Plains Public Library 100 Martine Avenue White Plains, New York 10601

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

- The licensee experienced several condenser tube leaks which required a controlled shutdown on February 14. Condenser repairs were completed and Unit returned to service on February 20.

FACILITY ITEMS (PLANS AND PROCEDURES):

- The licensee has completed his actions to comply with the thirty day requirements of the Confirmatory Order issued February 11, 1980.

PLANT STATUS:

- Unit at 100% Power.

- LAST IE SITE INSPECTION DATE: 3/29/80
- INSPECTION REPORT NO: 50-247/80-03

SUMMARY

INSPECTION SUMMARY

- 50-247/79-22 - Oct 22-26 and Oct 29-Nov 2: A special limited physical security inspection on the implementation and compliance with the accepted physical security plan relative to the security program audit, security organization, physical barriers (PA) (VA), lighting, access control, alarm stations, detection aids, assessment aids, communications, testing and maintenance, compensatory measures, power supply, response, locks, keys and combinations, records and reports. Followup on licensee action on previously identified items of noncompliance related to physical security. The inspection involved 107 inspector-hours onsite by one NRC regional based inspector and commenced during off-shift periods. Four items of noncompliance were identified (Failure to check vital area doors on Apr 15; Failure to check all security related doors on Sept 18; Failure to post security guard on compensatory security post on Apr 15; Failure to provide semi-annual and annual retraining for four guards).

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT

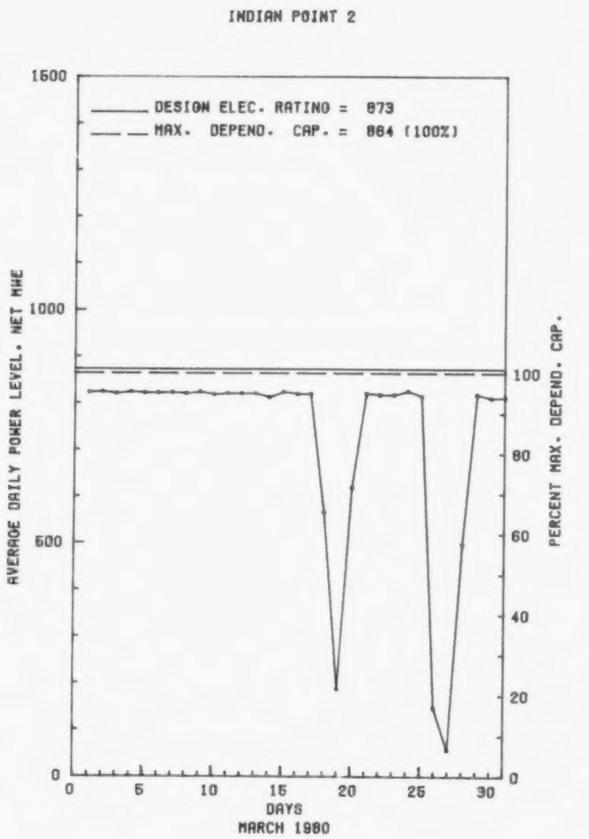
NONE.

REVIEWED BY OMPA LOEB: T. C. Cintula **DATE:** 5·9·80

REVIEWED BY NRR DOR: **DATE:** [illegible]

INDIAN POINT 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-247
2. Utility Contact: E. EICH (914) 696-6000 @ I.P.
3. Licensed Thermal Power (MWt): 2758
4. Nameplate Rating (Gross MWe): 1126 x 0.9 = 1013
5. Design Electrical Rating (Net MWe): 873
6. Maximum Dependable Capacity (Gross MWe): 900
7. Maximum Dependable Capacity (Net MWe): 864
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): 834
10. Reasons for Restrictions, If Any: REMOVAL OF #3 DISC ON #23 LOW PRES. TURBINE ROTOR.

ITEM 21 CALCULATED BY USING A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE*
11. Hours in Reporting Period	744.0	2,184.0	50,425.0
12. Hours Reactor Was Critical	712.7	1,322.0	33,616.5
13. Reactor Reserve Shutdown Hours	31.3	852.4	1,093.8
14. Hours Generator On-Line	707.4	1,266.7	32,655.9
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,851,710	3,206,325	84,038,851
17. Gross Electrical Energy (MWH)	566,130	956,450	26,089,536
18. Net Electrical Energy (MWH)	541,795	904,062	24,869,162
19. Unit Service Factor	95.1	58.0	64.8
20 Unit Availability Factor	95.1	58.0	64.8
*21. Unit Capacity Factor (MDC Net)	84.3	47.9	57.2
22. Unit Capacity Factor (DER Net)	83.4	47.4	56.5
23. Unit Forced Outage Rate	4.9	13.5	8.3

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

*Computed since 7/1/74; the date of completion of a 100 day – 100% power operation test.

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	03/18/80	F	.0	A	5		CH	VALVEX	LOAD REDUCTION CAUSED BY 22 FEEDWATER REGULATOR.
	03/19/80	F	.0	A	5		CH	VALVEX	LOAD REDUCTION CAUSED BY 22 FEEDWATER REGULATOR.
6	03/26/80	F	36.6	A	3		HJ	HTEXCH	NO. 22 S/G LOW LEVEL CONTROL AIR LINE TO HOT DUMP VALVE TO NO. 23 CONDENSER PARTED.
	03/27/80	F	.0	A	5		CH	PUMPXX	LOAD REDUCTION TO REPAIR 21 MAIN BOILER FEED PUMP.

SUMMARY

OPERATION WAS ROUTINE AT A POWER RESTRICTION TO 834 MWE (NET) DUE TO REMOVAL OF A DISC ON A LOW PRESSURE TURBINE ROTOR. LOAD REDUCTIONS OCCURRED ON 03/18/80 AND 03/19/80 TO REPAIR A MAIN FEEDWATER REGULATING VALVE, AND ON 03/27/80 TO REPAIR A MAIN BOILER FEED PUMP. ON 03/26/80, THE UNIT TRIPPED ON LOW LEVEL IN THE NO. 22 STEAM GENERATOR.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

INDIAN POINT 3

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	New York/Westchester 25 miles N of New York City
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	3025
5.	Design Electrical Rating (MWE Net)	965
6.	Date of Initial Criticality	April 6, 1976
7.	Date of Commercial Operation	August 30, 1976
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Hudson River

Utility & Contractor Information

10.	Licensee	Power Authority of the State of New York, Inc.
11.	Corporate Address	10 Columbus Circle New York, New York
12.	Corporate Contact	G.T. Barry General Manager
13.	Architect/Engineer	UE&C
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	WEDCO

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Senior Inspector	R. Keimig
19.	IE Resident Inspector	T. Rebelowski (914-739-9360) FTS 656-9220
20.	Licensing Project Manager	L. Olshan x27144
21.	Docket Number	50-286
22.	License Number/Date of Issue	DPR-64, 12/12/75
23.	Public Document Room	White Plains Public Library 100 Martine Avenue White Plains, New York 10601

INSPECTION STATUS

ENFORCEMENT STATUS

TS 6.8.1.B, LICFNSEE FAILED TO ADHERE TO WRITTEN PROCEDURES WHICH REQUIRED THE VENTING OF SPRAY HEADER PRIOR TO START OF CONTAINMENT SPRAY PUMP.
(7923 INFRACTION)

FAILURE TO ADHERE TO ADMINISTRATIVE PROCEDURES IN REGARD TO STORAGE OF COMBUSTIBLE MATERIALS IN VITAL AREAS. (T.S. 6.8.1.A.)
(7923 DEFICIENCY)

FAILURE TO USE A DOSE RATE MONITORING DEVICE WHEN ENTERING A HIGH RADIATION AREA IN PAB (PIPE PENETRATION AREA).
(7927 INFRACTION)

ON 12/13/79, AT APPROXIMATELY 10:30 A.M., THERE WAS AN UNATTENDED CONTAINER, A 55-GALLON DRUM USED IN VACUUMING CONTAMINATED LIQUIDS, ADJACENT TO THE MAIN PASSAGEWAY OF THE 55' ELEVATION OF THE PRIMARY AUXILIARY BUILDING, WHICH CONTAINED LICENSED MATERIAL & DID NOT BEAR A LABEL IDENTIFYING THE RADIOACTIVE CONTENTS & THE WORDS "CAUTION, (OR DANGER) RADIOACTIVE MATERIAL." 10 CFR 20.203, "CAUTION SIGNS, SIGNALS & CONTROLS."
(7930 DEFICIENCY)

CONTRARY TO 10 CFR 50.59, ON 9/14/79, THE LIQUID WASTE DISPOSAL SYSTEM (A SYS DESCRIBED IN THE FSAR) WAS CHANGED, IN THAT THE FLOWPATH & LOCATION WAS CHANGED BY THE ADDITION OF A SUPPLEMENTAL WASTE HANDLING SYS, BUT A SAFETY EVALUATION PROVIDING THE DOCUMENTED BASES FOR DETERMINATION THAT THE CHANGE DID NOT INVOLVE AN UNREVIEWED SAFETY QUESTION WAS NOT PERFORMED.
(7930 INFRACTION)

ON 1/12 & 8/23/79, THE LICENSEE SHIPPED RADIOACTIVE MATERIALS, WHICH WERE GREATER THAN TYPE A QUANTITIES, IN PACKAGES THAT HAD NOT BEEN CLOSED IN ACCORDANCE WITH ADEQUATE LICENSEE PROCEDURES. 10 CFR 71.54, "ROUTINE DETERMINATIONS."
(7930 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

- The licensee has experienced several trips attributed to control oil problems on MBFP and turbine lube oil contamination. Unit was removed from line on 3/7/80 and oil cleanup was completed. Unit returned to service on 3/13/80.

FACILITY ITEMS (PLANS AND PROCEDURES):

- The licensee has completed his actions to comply with the thirty day requirements of the Confirmatory Order issued February 11, 1980.

MANAGERIAL ITEMS:

- The licensee has completed his actions in regard to IAL 80-03.

PLANT STATUS:

Operation at reduced power.

- LAST IE SITE INSPECTION DATE: 3/25/80
- INSPECTION REPORT NO: 50-286/80-04

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
NONE				

SUMMARY

INSPECTION SUMMARY

- 50-286/79-23 - Sep 30-Oct 27: Routine, unannounced inspection of plant operations including: log and record reviews; tours of the facility; observation of refueling activities; observation of maintenance activities; review and observation of steam generator supports; witnessing of surveillance testing; observation of physical security; Bulletin 79-14 observations; emergency planning operation and independant inspection effort in area of containment spray actuation. The inspection involved 75 inspector-hours by the resident inspectors. Three items of noncompliance were found (Failure to control access; Failure to adhere to procedures; Failure to remove identified combustibles).

- 50-286/79-25 - Oct 23-26, Oct 29-Nov 2 and Nov 13-16 and Nov 19-21: A special physical protection inspection relative to the implementation of the Physical Security Plan; Security Program Audit; Security Organization; Physical Barriers (Protected Area) (Vital Areas); Lighting; Access Control (Identification, Authorization and Badging) (Search) (Escorting) (Vital Areas); Alarm Stations; Detection Aids; Assessment Aids; Communications; Testing and Maintenance; Compensatory Measures; Power Supply; Response;

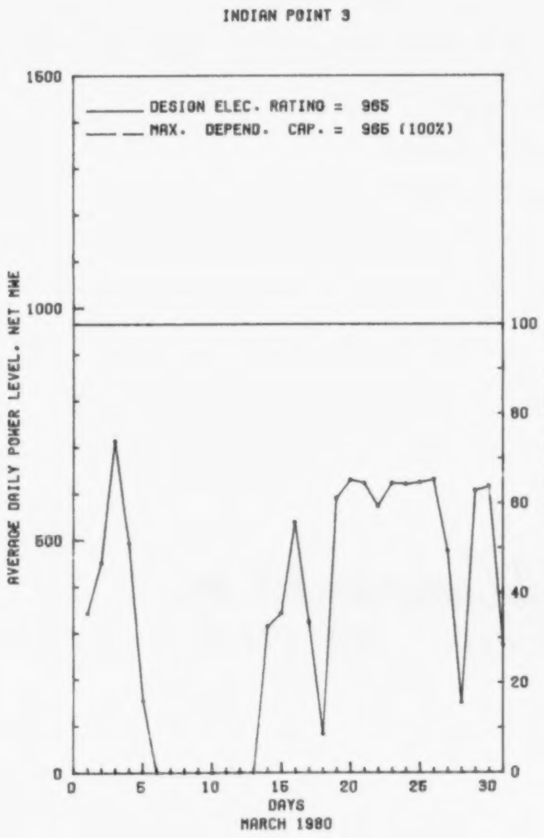
  Locks, Keys and Combinations and Records and Reports. The inspection involved 143 inspector-hours onsite by three NRC regional based inspectors. Also reviewed was the licensee's response to IE Bulletin 79-16. Each of these areas was examined in consideration of the allegations of lax security published in local newspapers. Four items of noncompliance were identified (Failure to maintain isolation zones; Failure to operate alarm station in accordance with requirements; Failure to perform detection aid maintenance; Failure to conduct training).

REVIEWED BY OMPA LOEB: T. C. Cintula **DATE:** 5·9·80

REVIEWED BY NRR DOR: Olshan **DATE:** 5-12-80

INDIAN POINT 3

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-286
2. Utility Contact: C. Connell (914) 739-8200 x242
3. Licensed Thermal Power (MWt): 3025
4. Nameplate Rating (Gross MWe): 1126 x 0.9 = 1013
5. Design Electrical Rating (Net MWe): 965
6. Maximum Dependable Capacity (Gross MWe): 1000
7. Maximum Dependable Capacity (Net MWe): 965
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
REMOVED TURBINE BLADING ON NO. 31 LP TURBINE
9. Power Level To Which Restricted, If Any (Net MWe): 912
10. Reasons for Restrictions, If Any:
TWO BLADING ROWS REMOVED ON L.P. TURBINE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	2,184.0	31,441.0
12. Hours Reactor Was Critical	563.5	1,093.4	22,678.3
13. Reactor Reserve Shutdown Hours	0	0	.0
14. Hours Generator On-Line	478.5	825.8	21,858.6
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	991,653	1,290,502	57,394,969
17. Gross Electrical Energy (MWH)	275,840	343,180	18,708,281
18. Net Electrical Energy (MWH)	258,929	316,264	17,959,701
19. Unit Service Factor	64.3	37.8	69.5
20. Unit Availability Factor	64.3	37.8	69.5
21. Unit Capacity Factor (MDC Net)	36.1	15.0	59.2
22. Unit Capacity Factor (DER Net)	36.1	15.0	59.2
23. Unit Forced Outage Rate	33.5	24.9	5.2

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
3	03/04/80	F	7.2	A	3		HH	PUMPXX	LOSS OF 31 MBFP-CLEAR CONTROL BLOCK
4	03/05/80	F	10.4	G	3		HB	VALVEX	33 MSIV INADVERTENTLY TRIPPED - INSTALLED CAGES.
5	03/05/80	F	4.9	A	3		HA	TURBIN	THRUST BEARING TURBINE TRIP - CLEANED DETECTOR.
6	03/06/80	F	188.6	B	3		HH	PUMPXX	31, 32, MBFP CONTROL SYSTEM FLUSHING.
7	03/17/80	S	24.7	B	1		HH	PIPEXX	MAIN FEEDWATER DRAIN LINE WELD LEAK - REPAIRED WELD.
8	03/22/80	F	.0	F	4		HF	FILTER	BLOCKAGE TO INTAKE SCREENS - CLEANED SCREENS.
9	03/27/80	F	15.7	G	3		HH	PUMPXX	AIR INTRODUCED INTO SYSTEM WHEN VALVING IN 31 CONDENSATE PUMP CAUSING STEAM FLOW/FEED-FLOW MISMATCH - INSTRUCTED OPERATORS.
10	03/28/80	F	3.8	A	3		HA	TURBIN	THRUST BEARING TURBINE TRIP - CLEANED DETECTORS.
11	03/31/80	F	10.2	A	3		IA	INSTRU	32 STATIC INVERTER FAILED. - INSPECTED INVERTER.

SUMMARY

UNIT TRIPS OCCURRED ON 03/04/80 FROM LOSS OF THE NO. 31 MAIN BOILER FEEDWATER PUMP; ON 03/05/80 WHEN A MAIN STEAM ISOLATION VALVE WAS ACCIDENTALLY TRIPPED; ON 03/27/80 FROM STEAM FLOW/FEED FLOW MISMATCH; AND ON 03/31/80 WHEN A STATIC INVERTER FAILED AND RESULTED IN ANOTHER STEAM FLOW/FEED FLOW MISMATCH. ON 03/05/80, A TURBINE THRUST BEARING TRIP WAS ALSO EXPERIENCED DUE TO A BLOCKAGE IN THE THRUST BEARING TRIP DETECTION DEVICE, AND A LOW STEAM GENERATOR LEVEL AND REACTOR TRIP OCCURRED ON STARTUP. A WELD LEAK IN A DRAIN LINE OFF THE MAIN FEEDWATER HEADER WAS REPAIRED ON 03/17/80, AND A SHUTDOWN OCCURRED ON 03/31/80 WHEN TECHNICAL SPECIFICATION TIME FOR THE NO. 31 AUXILIARY BOILER FEEDWATER PUMP WAS EXHAUSTED.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

KEWAUNEE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Wisconsin/Kewaunee 27 miles E of Green Bay
2.	Electric Reliability Council	Mid-America Interpool Network
3.	Type of Reactor	PWR
4.	Power Level (MWT)	1650
5.	Design Electrical Rating (MWE Net)	535
6.	Date of Initial Criticality	March 7, 1974
7.	Date of Commercial Operation	June 1974
8.	Condenser Cooling Method	Once through condenser and return to Lake Michigan
9.	Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10.	Licensee	Wisconsin Public Service Corp.
11.	Corporate Address	P. O. Box 1200 Green Bay, Wisconsin 54305
12.	Corporate Contact	E. R. Mathews Power Supply & Eng.
13.	Architect/Engineer	Fluor Power Services Corp.
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Pioneer Services & Eng. Co.

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Senior Inspector	R. F. Warnick
19.	IE Principal Inspector	N. C. Choules
20.	Licensing Project Manager	R. B. Licciardo X28362
21.	Docket Number	50-305
22.	License Number/Date of Issue	DPR-43 issued 12/21/73
23.	Public Document Room	Kewaunee Public Library 822 Juneau Street Kewaunee, Wisconsin 54216

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENTS

+ A bushing on the reserve auxiliary transformer failed on January 17, 1980 causing a plant shutdown. A fault in the main auxiliary transformer bus on January 4, 1980 had taken that transformer out of service and was out when the reserve auxiliary transformer failed. (80-02)

MANAGERIAL ITEMS

+ Effective March 1, 1980, Mr. D. C. Hintz, Nuclear Services Supervisor, Wisconsin Public Service, will become Kewaunee Nuclear Plant Superintendent. Mr. C. R. Luoma, present Plant Superintendent, is to be Assistant Manager, Nuclear Power at the Green Bay Corporate Office. (80-05)

PLANT STATUS

+ The plant operated routinely at power during the month.

+LAST IE SITE INSPECTION DATE: February 19-22, 1980

+ INSPECTION REPORT NO: 80-05

SUMMARY

INSPECTION SUMMARY

+ Inspection on January 17-22 and 25, (80-02): Special, announced inspection of a partial loss of station power and reactor trip. The inspection involved 61 inspector hours onsite by four NRC inspectors. One item of noncompliance was identified (lack of procedure for adding lubricating oil to the emergency diesel).

+ Inspection on February 4-5, (80-04): Licensee actions relative to IE Bulletins No. 79-14 and No. 79-04 including general discussion on NRC requirements, work procedure review and record review. The inspection involved 10 inspector hours onsite by one NRC inspector. No items of noncompliance or deviations were identified.

+ Inspection on February 19-22, (80-05): Routine, unannounced nuclear material control and accountability inspection of previously identified items, facility organizations and operation, measurement and controls, shipping and receiving, internal control, inventory, records and reports and management of control system. The inspection involved 42 inspector hours onsite by two NRC inspectors and was begun during the regular hours. The licensee was found to be in compliance with NRC requirements in the eight areas examined during the inspection.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

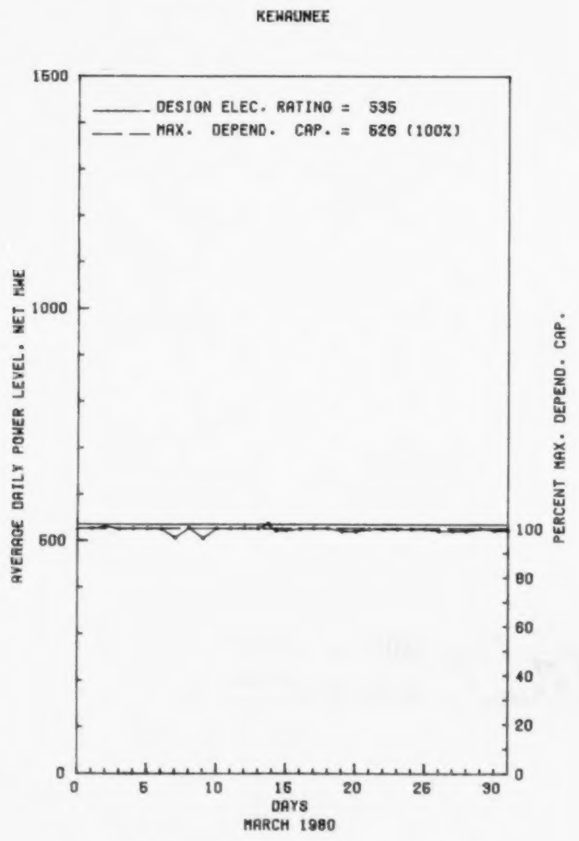
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
80-05/ 03L-0	02/01/80	02/28/80	30 Day LER	Shift instrument channel checks were not recorded as required by TS.
80-06/ 03L-0	02/02/80	02/28/80	30 Day LER	Condensate water storage tank level below TS limit.
80-07/ 03L-0	02/08/80	03/06/80	30 Day LER	S/G steam flow channel indicating low.
80-08/ 03L-0	02/11/80	03/10/80	30 Day LER	Charging pump failed to remain running when control switch was returned to Auto.
80-09/ 03L-0	02/20/80	03/21/80	30 Day LER	Small leak on the discharge of coolant charging pump.
80-10/ 03L-0	02/20/80	03/21/80	30 Day LER	Damper cycled in mid-position.
80-11/ 03L-0	02/20/80	03/21/80	30 Day LER	Containment radiation monitoring system pump indicated high flow and tripped off.
80-12/ 03L-0	02/21/80	03/21/80	30 Day LER	Diesel generator breaker indication lost and breaker would not close.
80-13/ 03L-0	02/23/80	03/24/80	30 Day LER	Charging pumps of out service.

==

REVIEWED BY OMPA LOEB: T. C. Cintula **DATE:** 5-9-80

REVIEWED BY NRR DOR: R. Licciardo **DATE:** 5-12-80

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-305
2. Utility Contact: G. Ruiter (414) 388-2560 x207
3. Licensed Thermal Power (MWt): 1650
4. Nameplate Rating (Gross MWe): 622 x 0.9 = 560
5. Design Electrical Rating (Net MWe): 535
6. Maximum Dependable Capacity (Gross MWe): 545
7. Maximum Dependable Capacity (Net MWe): 526
8. If Changes Occur in Capacity Ratings Above Since Last Report. Give Reasons:
NONE
9. Power Level To Which Restricted. If Any (Net MWe): NONE
10. Reasons for Restrictions. If Any:
NONE

ITEM 21 CALCULATED BY USING A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	2,184.0	50,785.0
12. Hours Reactor Was Critical	744.0	1,917.0	42,919.5
13. Reactor Reserve Shutdown Hours	.0	152.7	2,330.5
14. Hours Generator On-Line	744.0	1,899.8	41,895.8
15. Unit Reserve Shutdown Hours	.0	.0	10.0
16. Gross Thermal Energy (MWH)	1,223,050	3,060,998	64,212,121
17. Gross Electrical Energy (MWH)	407,700	1,020,200	21,204,000
18. Net Electrical Energy (MWH)	389,565	974,556	20,173,634
19. Unit Service Factor	100.0	87.0	82.5
20. Unit Availability Factor	100.0	87.0	82.5
*21. Unit Capacity Factor (MDC Net)	99.5	84.8	75.8
22. Unit Capacity Factor (DER Net)	97.9	83.4	74.2
23. Unit Forced Outage Rate	.0	13.0	5.7

24. Shutdowns Scheduled Over Next 6 Months (Type.Date.and Duration of Each):
REFUELING; MAY 10, 1980; SIX WEEKS.
25. If Shut Down At End of Report Period. Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	NONE								

SUMMARY

OPERATION WAS AT NEAR FULL POWER DURING THE REPORT PERIOD. WITH BRIEF REDUCTIONS FOR REPAIR OF THE HEATER DRAIN PUMP 1A, AND TO BRING CIRCULATING WATER DISCHARGE TEMPERATURE WITHIN TECHNICAL SPECIFICATION LIMITS.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram.
3--Automatic Scram.
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

H:
urce.

LACROSSE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Wisconsin/Vernon 19 miles S of LaCrosse
2.	Electric Reliability Council	Mid-Continent Area Reliability Coordination Agreement
3.	Type of Reactor	BWR
4.	Power Level (MWT)	165
5.	Design Electrical Rating (MWE Net)	50
6.	Date of Initial Criticality	July 11, 1967
7.	Date of Commercial Operation	November 1, 1969
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Mississippi River

Utility & Contractor Information

10.	Licensee	Dairyland Power Cooperative
11.	Corporate Address	2615 East Avenue South La Crosse, Wisconsin
12.	Corporate Contact	F. Linder
13.	Architect/Engineer	Sargent & Lundy
14.	Nuclear Steam Supply System	Allis-Chalmers
15.	Constructor	Maxson Construction

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	D. C. Boyd
18.	IE Senior Inspector	R. F. Warnick
19.	IE Principal Inspector	K. Ridgway
20.	Licensing Project Manager	J. J. Shea
21.	Docket Number	50-409
22.	License Number/Date of Issue	DPR-45 issued 8/28/73
23.	Public Document Room	LaCrosse Public Library 800 Main Street LaCrosse, Wisconsin 54601

INSPECTION STATUS

ENFORCEMENT STATUS

TRAINING WAS NOT CONDUCTED IN ACCORDANCE WITH SECURITY PLAN COMMITMENTS.
(7923 INFRACTION)

REQUIRED SECURITY EQUIPMENT WAS NOT BEING UTILIZED.
(7923 DEFICIENCY)

THE LICENSEE FAILED TO PROPERLY CONTROL PICTURE IDENTIFICATION BADGES.
(7923 DEFICIENCY)

THE LICENSEE FAILED TO PROPERLY CONTROL ACCESS.
(7923 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

Plant is operating routinely.

*LAST IE SITE INSPECTION DATE: December 10-14, 1979

* INSPECTION REPORT NO: 79-23

INSPECTION SUMMARY

* Inspection on December 10-14, (79-23): Included a review of Security Plan; Security Program Audit; Security Organization; Protected Area Physical Barriers; Vital Area Physical Barriers; Lighting; Access Control (Identification, Authorization and Badging); Access Control (Search); Access Control (Escorting); Access Control (Vital Areas); Alarm Station; Detection Aids; Assessment Aids; Communications; Testing and Maintenance; Compensatory Measures; Power Supply; Response; Locks, Keys, and Combinations; Records and Reports; Followup on Previous Item of Noncompliance; Followup on IE Bulletin No. 79-16. The insepction involved 24 inspector hours onsite by two NRC inspectors. The inspection began during the day shift; 24 inspection hours were accomplished during offshift periods. The licensee was found to be in compliance with NRC requirements within the 22 areas examined during this inspection except for the following four items: (A) Security Organization - Training was not conducted in accordance with security plan commitments; (B) Security Organization - Required security equipment was not being utilized; (C) Access Control (Identification, Authorization and Badging) - The licensee failed to properly control pictured identification badges; (D) Access Control (Search) - The licensee failed to properly control access. Two Unresolved Items were identified. These items will be referred to NRC Headquarters for evaluation and resolution. (A) The licensee relocated an assessment aid; and (B) The licensee did not follow a security plan commitment concerning the protection of one central communication system.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

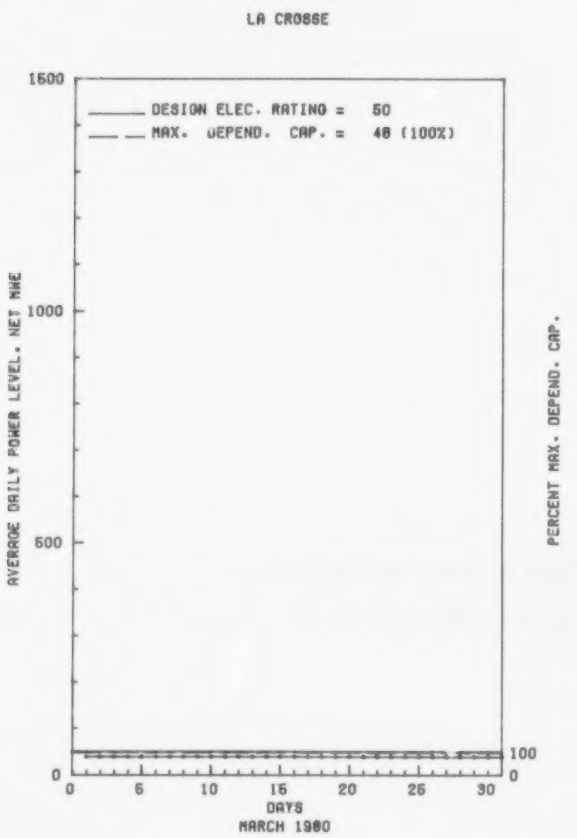
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
NONE				

REVIEWED BY OMPA LOEB: John J. Rizzo DATE: 5-12-80

REVIEWED BY NRR DOR: J J Shea DATE: 5-13-80

LACROSSE

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-409
2. Utility Contact: L. S. Goodman (608) 689-2331
3. Licensed Thermal Power (MWt): 165
4. Nameplate Rating (Gross MWe): 76.8 x 0.85 =65
5. Design Electrical Rating (Net MWe): 50
6. Maximum Dependable Capacity (Gross MWe): 50
7. Maximum Dependable Capacity (Net MWe): 48
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	2,184.0	91,299.0
12. Hours Reactor Was Critical	744.0	2,126.1	59,932.2
13. Reactor Reserve Shutdown Hours	.0	.0	478.0
14. Hours Generator On-Line	744.0	2,109.5	55,114.5
15. Unit Reserve Shutdown Hours	.0	.0	79.0
16. Gross Thermal Energy (MWH)	102,303	290,880	7,575,659
17. Gross Electrical Energy (MWH)	29,903	85,843	2,284,975
18. Net Electrical Energy (MWH)	28,197	80,844	2,113,579
19. Unit Service Factor	100.0	96.6	60.4
20. Unit Availability Factor	100.0	96.6	60.5
21. Unit Capacity Factor (MDC Net)	79.0	77.1	48.2
22. Unit Capacity Factor (DER Net)	75.8	74.0	46.3
23. Unit Forced Outage Rate	.0	3.4	6.3

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): EQUIP INSTALL (NUREG-0578) 4/6/80, - REFUELING - 8/18/80
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-02	03/20/80	F	.0	B	5		HC	XXXXXX	A POWER REDUCTION FROM 85% TO 47% WAS CONDUCTED FOR PURPOSE OF REMOVING EACH OF TWO TURBINE CONDENSER WATER BOXES TO CLEAN CONDENSER TUBES AND TO REMOVE ACCUMULATED SILT FROM THE WATER BOXES. CONDENSER CLEANING WAS ACCOMPLISHED TO IMPROVE TURBINE EXHAUST VACUUM AND TEMPERATURE CONDITIONS.

SUMMARY

OPERATION WAS ROUTINE AT 85% RATED THERMAL POWER TO EXTEND CORE LIFE. LOAD WAS REDUCED ON 03/20/80 FOR CONDENSER WATERBOX CLEANING.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

MAINE YANKEE

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1. Location (state/county) (nearest population center)	Maine/Lincoln 10 miles N of Bath (near Wiscasset)
2. Electric Reliability Council	Northeast Power Coordinating Council
3. Type of Reactor	PWR
4. Power Level (MWT)	2630
5. Design Electrical Rating (MWE Net)	825
6. Date of Initial Criticality	October 23, 1972
7. Date of Commercial Operation	December 28, 1972
8. Condenser Cooling Method	Once Through
9. Condenser Cooling Water Source	Back River (Atlantic)

Utility & Contractor Information

10. Licensee	Maine Yankee Atomic Power Co.
11. Corporate Address	20 Turnpike Road Westboro, Massachusetts 01581
12. Corporate Contact	D. W. Edwards, Assistant to Vice President
13. Architect/Engineer	Stone & Webster
14. Nuclear Steam Supply System	Combustion Engineering
15. Constructor	Stone & Webster

Regulatory Information

16. IE Region responsible	I
17. IE Branch Chief	E. Brunner
18. IE Senior Inspector	T. T. Martin
19. IE Principal Inspector	W. J. Lazarus
20. Licensing Project Manager	M. B. Fairtile
21. Docket Number	50-309
22. License Number/Date of Issue	DPR-36, 9/15/72
23. Public Document Room	Wiscasset Public Library High Street Wiscasset, Maine 04578

INSPECTION STATUS

ENFORCEMENT STATUS

FAILED TO FOLLOW PROCEDURE 10-1 WHICH REQUIRED AUDIBLE NEUTRON COUNT RATE IN REFUELING CAVITY AREA WHEN MAKING CORE ALTERATIONS.
(8001 INFRACTION)

FAILED TO ADHERE TO REQUIREMENTS OF A RADIATION WORK PERMIT AS REQUIRED BY PROCEDURE 9.1.10.
(8001 INFRACTION)

NO PROCEDURE FOR THE OPERATION OF THE BALING MACHINE USED TO COMPACT TRASH IN 4 FT BY 4 FT BY 8 FT BOXES HAD BEEN ESTABLISHED, REVIEWED & IMPLEMENTED ALTHOUGH THE MACHINE WAS IN USE DURING THE WEEK OF 1/14-18/80. TS 5.8.1.
(8002 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS:

- Low power physics testing and power escalation following startup from refueling on March 9.
- LAST IE SITE INSPECTION DATE: 3/21/80
- INSPECTION REPORT NO: 50-309/80-05

SUMMARY

INSPECTION SUMMARY

- 50-309/80-03 - Jan 22-25: Routine, unannounced inspection of the licensee inspection program and the implementation of the examination and testing requirements imposed by ASME Section XI. The inspection consists of program review, personnel qualification, observation of N.D.E. performed and review of test data. The inspection involved 26 inspection hours onsite by one NRC region-based inspector. No items of noncompliance were identified.
- 50-309/80-04 - Feb 18-22, 27 and Mar 4-7: Routine, unannounced inspection of refueling activities; review of plant organization and administration; followup on selected Licensee Event Reports (LER's); and participation in a meeting with representatives of the licensee and NRC's Office of Nuclear Reactor Regulation concerning implementation of Short-Term Lessons Learned resulting from Three Mile Island 2. The inspection involved 69 inspector-hours onsite by two region-based inspectors. Two items of noncompliance were identified (Infraction - No procedure detailing the responsibilities and authorities of plant staff members; and, Infraction - cleanup of Reactor Coolant System from sodium chloride injection incident without a procedure).

REPORTS RECEIVED FROM LICENSEE

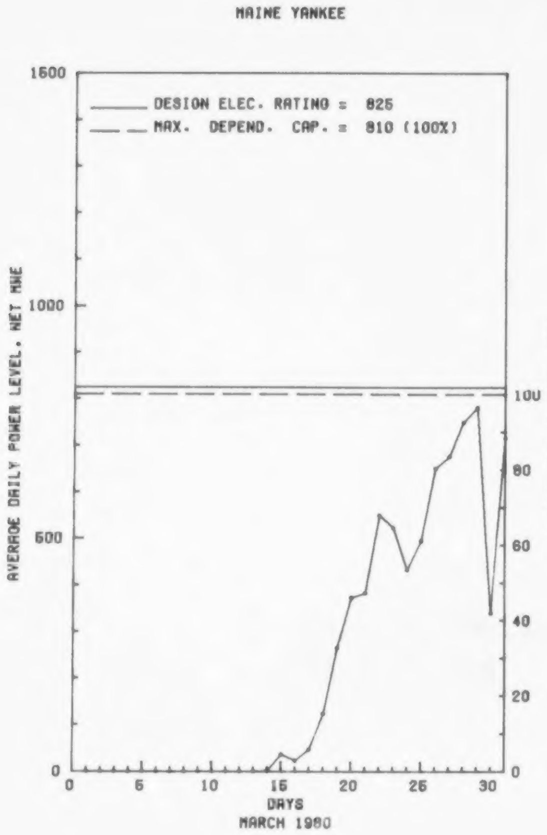
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
80-001/01T	01/30/80	03/21/80	14 Day LER	Main steam line break safety analysis error
80-005/03L	01/31/80	03/01/80	30 Day LER	Cracked safety injection header drain line

REVIEWED BY OMPA LOEB: JmL Beeler DATE: 05/12/80

REVIEWED BY NRR DOR: A. Segna DATE: 05/12/80

MAINE YANKEE

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-309
2. Utility Contact: Robert H. Groce (617) 366-9011
3. Licensed Thermal Power (MWt): 2630
4. Nameplate Rating (Gross MWe): 900 x 0.9 = 810
5. Design Electrical Rating (Net MWe): 825
6. Maximum Dependable Capacity (Gross MWe): 850
7. Maximum Dependable Capacity (Net MWe): 810
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

CAPACITY FACTORS CALCULATED WITH A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	2,184.0	64,812.6
12. Hours Reactor Was Critical	506.2	769.6	51,979.5
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	379.3	642.4	50,160.1
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	573,590	998,089	106,240,172
17. Gross Electrical Energy (MWH)	182,880	322,540	34,895,720
18. Net Electrical Energy (MWH)	172,033	303,073	33,130,259
19. Unit Service Factor	51.0	29.4	77.4
20. Unit Availability Factor	51.0	29.4	77.4
21. Unit Capacity Factor (MDC Net)	28.5	17.1	66.4
22. Unit Capacity Factor (DER Net)	28.0	16.5	64.1
23. Unit Forced Outage Rate	13.1	8.2	6.9

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
8-79-4	03/09/80	S	303.3	C	1				CYCLE 4/5 REFUELING SHUTDOWN.
1-80-5	03/10/80	S	2.8	B	1				SHUTDOWN WAS A PART OF LOW POWER PHYSIC TESTING. GENERATOR HAD NOT YET BEEN PHASED TO THE GRID.
2-80-5	03/12/80	F	28.6	A	3		IA	INSTRU	DURING ROUTINE RPS SURVEILLANCE TESTING BROKEN WIRE IN THE RPS LOGIC LADDER CAUSED A REACTOR TRIP. THE WIRE WAS REPAIRED AND RETURNED TO SERVICE. THE GENERATOR HAD NOT YET BEEN PHASED TO THE GRID SINCE THE STARTUP OF CYCLE 5.
3-80-5	03/12/80	S	1.2	E	1				REACTOR OPERATOR TRAINING.
	03/15/80	F	8.9	F	1			TURBIN	UNIT TAKEN OFF-LINE TO ADD A BALANCE WEIGHT.
	03/16/80	F	8.0	F	1			TURBIN	UNIT TAKEN OFF-LINE TO ADD A BALANCE WEIGHT.
	03/16/80	F	6.1	F	1			TURBIN	UNIT TAKEN OFF-LINE TO ADD A BALANCE WEIGHT.
	03/17/80	F	5.8	F	1			TURBIN	UNIT TAKEN OFF-LINE TO ADD A BALANCE WEIGHT.
	03/26/80	F	63.7	A	5		CC	ZZZZZZ	LOAD REDUCTION TO 55% POWER. INCREASING CHLORIDE CONCENTRATION IN THE STEAM GENERATOR DUE TO CONDENSER IN LEAKAGE PLUS CHANGING THE LUBE OIL IN P-2B FEED PUMP DUE TO EXCESSIVE DIRT.
	03/31/80	F	49.0	H	5		CC	ZZZZZZ	LOAD REDUCTION TO 20% POWER TO HELP LEACH RESIDUAL CHLORIDES OUT OF THE STEAM GENERATORS.

SUMMARY

THE UNIT WAS BROUGHT ON LINE ON 03/14/80 FOLLOWING AN OUTAGE FOR REFUELING; LOW POWER PHYSICS TESTING; CONDENSATE, FEEDWATER, AND STEAM GENERATOR CLEANUP; AND REACTOR OPERATOR TRAINING. POWER ESCALATION CONTINUED THROUGH THE END OF THE REPORT PERIOD, WITH LOAD REDUCTIONS FOR CONDENSER TUBE LEAK REPAIR AND A STEAM GENERATOR CHLORIDE SOAK.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram.
3--Automatic Scram.
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

MILLSTONE 1

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Connecticut/New London 5 miles SW of New London
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2011
5.	Design Electrical Rating (MWE Net)	660
6.	Date of Initial Criticality	October 26, 1970
7.	Date of Commercial Operation	March 1971
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Long Island Sound

Utility & Contractor Information

10.	Licensee	Northeast Nuclear Energy Company
11.	Corporate Address	P. O. Box 270 Hartford, Connecticut
12.	Corporate Contact	W. G. Counsil, Vice President
13.	Architect/Engineer	Ebasco
14.	Nuclear Steam Supply System	GE
15.	Constructor	Ebasco

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	IE Senior Inspector	E. McCabe (FTS: 244-2000)
19.	IE Resident Inspector	J. Shedlosky (203) 447-3179
20.	Licensing Project Manager	J. J. Shea
21.	Docket Number	50-245
22.	License Number/Date of Issue	DPR-21, 10/7/70
23.	Public Document Room	Waterford Public Library Rope Ferry Road Route 156 Waterford, Connecticut 06385

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

- The core spray subsystem 'A' was considered operable on March 7, following repairs to damaged subsystem pipe restraints and modification to the keep-full system in order to prevent further water hammer.

PLANT STATUS:

- The unit operated at a nominal 100% power during the month. Representative offgas and stack release rates were 135,000 and 320 microcuries per second, respectively.

- LAST IE SITE INSPECTION DATE: 3/31/80

- INSPECTION REPORT NO: 50-245/80-04

SUMMARY

INSPECTION SUMMARY

- 50-245/80-01 - Jan 1-31: Routine, onsite regular and backshift inspection by two resident inspectors (67 hours). Areas inspected included the control room and the accessible portions of the reactor, turbine, radioactive waste, gas turbine generator, and intake buildings; radiation protection; physical security; fire protection; plant operating records; surveillance testing; calibration; maintenance; core power distribution limits; and reporting to the NRC. Noncompliances: None.

- 50-245/80-03 - Feb 1-29: Routine, onsite regular and backshift inspection by two resident inspectors (122 hours). Areas inspected included the control room and the accessible portions of the reactor, turbine, radioactive waste, gas turbine generator, and intake buildings; radiation protection; physical security; fire protection; plant operating records; surveillance testing; calibration; maintenance; core power distribution limits; and reporting to the NRC. Noncompliances: None.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

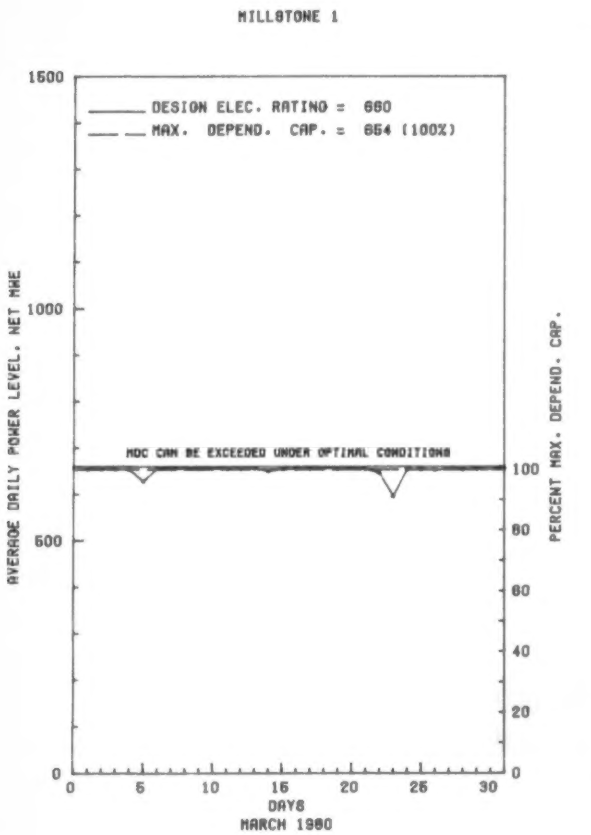
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
80-004/ 03L	01/21/80	02/14/80	30 Day LER	One of sixteen main steam line high flow switches found set above T.S. requirements
80-005/ 03L	02/20/80	03/17/80	30 Day LER	Core spray subsystem 'A' placed out of service to repair pipe supports damages by water hammer and to modify the keep-fill system to prevent recurrence

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 5-12-80

REVIEWED BY NRR DOR: J J Shea **DATE:** 5-13-80

MILLSTONE 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-245
2. Utility Contact: R. Young (203) 447-1792 x475
3. Licensed Thermal Power (MWt): 2011
4. Nameplate Rating (Gross MWe): 735 x 0.9 = 662
5. Design Electrical Rating (Net MWe): 660
6. Maximum Dependable Capacity (Gross MWe): 684
7. Maximum Dependable Capacity (Net MWe): 654
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	2,184.0	81,864.0
12. Hours Reactor Was Critical	744.0	2,184.0	62,569.0
13. Reactor Reserve Shutdown Hours	.0	.0	935.0
14. Hours Generator On-Line	744.0	2,184.0	60,279.9
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,481,527	3,264,183	107,312,128
17. Gross Electrical Energy (MWH)	507,400	1,099,300	36,330,696
18. Net Electrical Energy (MWH)	485,221	1,048,376	34,688,158
19. Unit Service Factor	100.0	100.0	73.6
20. Unit Availability Factor	100.0	100.0	73.6
21. Unit Capacity Factor (MDC Net)	99.7	73.4	64.8
22. Unit Capacity Factor (DER Net)	98.8	72.7	64.2
23. Unit Forced Outage Rate	.0	.0	16.1

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	NONE								

SUMMARY

OPERATION WAS AT NEAR FULL POWER DURING THE REPORT PERIOD, WITH VERY BRIEF REDUCTIONS FOR MINOR MAINTENANCE AND SCHEDULED TESTING.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

MILLSTONE 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Connecticut/New London 5 miles SW of New London
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2700
5.	Design Electrical Rating (MWE Net)	870
6.	Date of Initial Criticality	October 17, 1975
7.	Date of Commercial Operation	December 26, 1975
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Long Island Sound

Utility & Contractor Information

10.	Licensee	Northeast Nuclear Energy Company
11.	Corporate Address	P. O. Box 270 Hartford, Connecticut
12.	Corporate Contact	W. G. Counsil, Vice President
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Combustion Engineering
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	IE Senior Inspector	E. McCabe
19.	IE Resident Inspector	(FTS: 244-2000) J. Shedlosky (203) 447-3179 R. Zimmerman
20.	Licensing Project Manager	E. L. Conner, Jr.
21.	Docket Number	50-336
22.	License Number/Date of Issue	DPR-65, 8/1/75
23.	Public Document Room	Waterford Public Library Rope Ferry Road Route 156 Waterford, Connecticut 06385

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

+ Replacement charging pumps found to have cylinder scoring #2 feed regulating valve malfunctioned during trip on 3/12. Valve cylinder replaced due to scoring.

PLANT STATUS:

+ With the exception of reactor trips on March 12, 21 and 23, the unit operated at a nominal 100% power.

+ LAST IE SITE INSPECTION DATE: 3/31/80

+ INSPECTION REPORT NO: 50-336/80-03

SUMMARY

INSPECTION SUMMARY

+ 50-336/80-01 - Jan 1-31: Routine, onsite regular and backshift inspection by two resident inspectors (20.5 hours). Areas inspected included the control room and the accessible portions of the enclosure, auxiliary, turbine and intake buildings, and the condensate polishing facility; radiation protection; physical security; fire protection; plant operating records; surveillance testing; calibration; maintenance; core power distribution limits; and reporting to the NRC. Noncompliances: None.

+ 50-336/80-02 - Feb 1-29: Routine, onsite regular and backshift inspection by two resident inspectors (84 hours). Areas inspected included the control room and the accessible portions of the enclosure, auxiliary, turbine and intake buildings, and the condensate polishing facility; surveillance testing; calibration; maintenance; core power distribution limits; and reporting to the NRC. Noncompliances: None.

REPORTS RECEIVED FROM LICENSEE

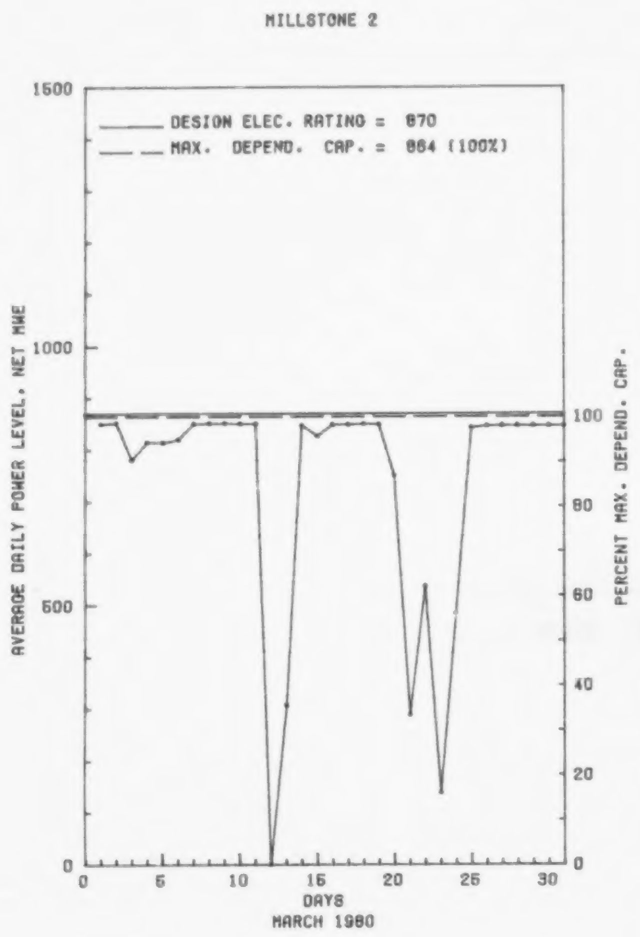
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
80-002/ 03L	01/31/80	02/20/80	30 Day LER	Packing leak on HPSI header isolation valve. Valve removed from service for repairs for 11 hours and 22 minutes
80-003/ 03L	01/22/80	02/21/80	30 Day LER	Containment air temperature .8 degrees above T.S. limit of 120 degrees for approximately 8 hours
80-004/ 03L	02/04/80	03/03/80	30 Day LER	RPS Channel B trip signal for reactor coolant pump underspeed found to be out of specification high

REVIEWED BY OMPA LOEB: M R Beebe **DATE:** 06/12/80

REVIEWED BY NRR DOR: E L Conner **DATE:** 5/12/80

MILLSTONE 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-336
2. Utility Contact: G. H. Howlett (203) 447-1791 x364
3. Licensed Thermal Power (MWt): 2700
4. Nameplate Rating (Gross MWe): 1011 x 0.9 = 910
5. Design Electrical Rating (Net MWe): 870
6. Maximum Dependable Capacity (Gross MWe): 895
7. Maximum Dependable Capacity (Net MWe): 864

CAPACITY FACTORS CALCULATED WITH A WEIGHTED AVERAGE

8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	2,184.0	37,392.0
12. Hours Reactor Was Critical	698.5	2,112.6	27,191.3
13. Reactor Reserve Shutdown Hours	.0	.0	2,072.4
14. Hours Generator On-Line	678.0	2,059.1	25,879.3
15. Unit Reserve Shutdown Hours	.0	.0	335.4
16. Gross Thermal Energy (MWH)	1,740,751	5,425,544	63,124,172
17. Gross Electrical Energy (MWH)	567,033	1,778,469	20,363,114
18. Net Electrical Energy (MWH)	544,231	1,710,893	19,491,151
19. Unit Service Factor	91.1	94.3	69.2
20. Unit Availability Factor	91.1	94.3	70.1
21. Unit Capacity Factor (MDC Net)	84.7	90.7	63.6
22. Unit Capacity Factor (DER Net)	84.1	90.0	62.3
23. Unit Forced Outage Rate	8.9	5.7	22.6

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
REFUELING (CYCLE 4), JULY 1, 1980, 9 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWN/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
4	03/12/80	F	29.0	A	3		HH	VALVEX	FEEDWATER REGULATING VALVE (2-FW-51B) LOCKED-UP (CLOSED), RESULTING IN A REACTOR TRIP ON LOW STEAM GENERATOR LEVEL.
5	03/20/80	F	.0	A	5		RB	CRDRVE	DROPPED C.E.A. #7-1 DUE TO FAILED CONTROL ELEMENT DRIVE MECHANISM POWER SUPPLY. REDUCED POWER TO 65%.
6	03/21/80	F	18.3	B	3		ZZ	ZZZZZZ	INSTRUMENTATION TECHNICIAN REMOVED MAIN FEED PUMP SUCTION PRESSURE TRANSMITTER FROM SERVICE FOR CALIBRATION. THIS LEAD TO THE PUMP TRIPPING ON LOW SUCTION PRESSURE AND A SUBSEQUENT REACTOR TRIP ON LOW STEAM GENERATOR LEVEL.
7	03/23/80	F	18.7	A	3		HH	VALVEX	FEEDWATER REGULATING VALVE (2-FW-51A) LOCKED-UP (CLOSED) DURING A POWER REDUCTION FOR A FEEDWATER HEATER PROBLEM. THE REACTOR TRIPPED FROM 20% POWER ON LOW STEAM GENERATOR LEVEL.

SUMMARY

REACTOR TRIPS OCCURRED ON 03/12/80 AND 03/23/80 WHEN A FEEDWATER REGULATING VALVE LOCKED-UP, AND ON 03/21/80 ON LOW STEAM GENERATOR LEVEL. POWER WAS REDUCED ON 03/20/80 WHEN A CONTROL ELEMENT ASSEMBLY DROPPED DUE TO FAILED CONTROL ELEMENT DRIVE MECHANISM POWER SUPPLY.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

MONTICELLO

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Minnesota/Wright 30 miles NW of Minneapolis
2.	Electric Reliability Council	Mid-Continent Area Reliability Coordination Agreement
3.	Type of Reactor	BWR
4.	Power Level (MWT)	1670
5.	Design Electrical Rating (MWE Net)	545
6.	Date of Initial Criticality	December 10, 1970
7.	Date of Commercial Operation	June 30, 1971
8.	Condenser Cooling Method	Once Through with Cooling Tower
9.	Condenser Cooling Water Source	Mississippi River

Utility & Contractor Information

10.	Licensee	Northern States Power Co.
11.	Corporate Address	414 Nicollet Mall - 8th Floor Minneapolis, Minnesota 55401
12.	Corporate Contact	G. H. Neils (Oper.) L. O. Mayer (Lic.)
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	GE
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heisman
18.	IE Senior Inspector	R. F. Warnick
19.	IE Principal Inspector	N. C. Choules
20.	Licensing Project Manager	T. Kevern
21.	Docket Number	50-263
22.	License Number/Date of Issue	DPR-22 issued 9/8/70
23.	Public Document Room	Environmental Conservation Library Minneapolis Public Library 300 Nicollet Mall Minneapolis, Minnesota 55401

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

Shut down for refueling.

+LAST IE SITE INSPECTION DATE: February 4-8, 1980

+ INSPECTION REPORT NO: 80-03

SUMMARY

INSPECTION SUMMARY

+ Inspection on February 4-8, (80-03): Included a review of Security Plan; Security Program Audit; Security Organization; Protected Area Physical Barriers; Vital Area Physical Barriers; Lighting; Access Control (Identification, Authorization and Badging); Access Control (Search); Access Control (Escorting); Access Control (Vital Areas); Alarm Station; Detection Aids; Assessment Aids; Communications; Testing and Maintenance; Compensatory Measures; Power Supply; Response; Locks, Keys, and Combinations; Records and Reports; Followup on IE Bulletin No. 79-16. The inspection involved 109.5 inspector hours onsite or at the licensee's corporate office by two NRC inspectors and one inspection assistant. Of the areas inspected, one item of noncompliance was identified in the area of physical barriers (Vital Area). The inadequacy was repaired satisfactorily prior to the conclusion of this inspection, and the item is considered closed.

REPORTS RECEIVED FROM LICENSEE

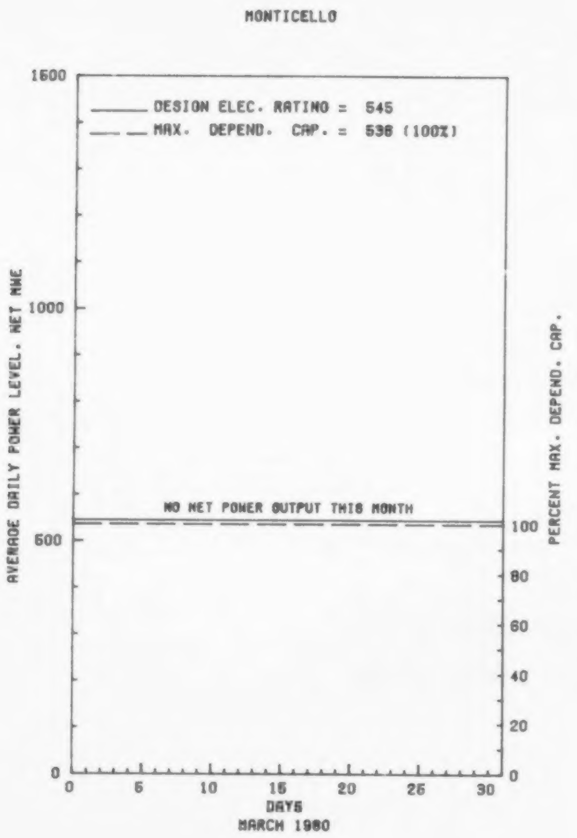
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
80-06/ 03L-0	02/26/80	03/26/80	30 Day LER	Main steam line area temperature switch setpoint drift.
80-07/ 01T-0	03/03/80	03/17/80	14 Day LER	HPCI-ADS cable separation.
80-11/ 01T-0	03/06/80	03/20/80	14 Day LER	Procedure inadequacy - MSIV maintenance.
80-13/ 01T-0	03/09/80	03/17/80	14 Day LER	Refueling rod block interlock failure.

REVIEWED BY OMPA LOEB: John J. [illegible] **DATE:** 5-12-80

REVIEWED BY NRR DOR: **DATE:** 5/13/80

MONTICELLO

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-263
2. Utility Contact: A. L. Myrabo (612) 295-5151
3. Licensed Thermal Power (MWt): 1670
4. Nameplate Rating (Gross MWe): 632 x 0.9 = 569
5. Design Electrical Rating (Net MWe): 545
6. Maximum Dependable Capacity (Gross MWe): 564
7. Maximum Dependable Capacity (Net MWe): 536
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	2,184.0	76,729.0
12. Hours Reactor Was Critical	0	1,223.6	62,754.5
13. Reactor Reserve Shutdown Hours	0	.0	940.7
14. Hours Generator On-Line	0	1,211.1	61,181.2
15. Unit Reserve Shutdown Hours	0	.0	.0
16. Gross Thermal Energy (MWH)	0	1,553,878	100,495,740
17. Gross Electrical Energy (MWH)	0	521,150	31,499,210
18. Net Electrical Energy (MWH)	-2,238	494,842	30,127,046
19. Unit Service Factor	.0	55.5	79.7
20. Unit Availability Factor	.0	55.5	79.7
21. Unit Capacity Factor (MDC Net)	.0	42.3	73.3
22. Unit Capacity Factor (DER Net)	.0	41.6	72.0
23. Unit Forced Outage Rate	.0	.7	6.4

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 04/04/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
4	02/22/80	S	744.0	C	4		ZZ	ZZZZZZ	CONTINUATION OF 1980 REFUELING OUTAGE.

SUMMARY

THE UNIT REMAINED SHUTDOWN THROUGHOUT THE REPORT PERIOD FOR REFUELING.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

NINE MILE POINT 1

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	New York/Oswego 8 miles NE of Oswego
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	1850
5.	Design Electrical Rating (MWE Net)	620
6.	Date of Initial Criticality	September 5, 1969
7.	Date of Commercial Operation	December 1969
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Lake Ontario

Utility & Contractor Information

10.	Licensee	Niagara Mohawk Power Corporation
11.	Corporate Address	300 Erie Boulevard West Syracuse, New York 13202
12.	Corporate Contact	R. Schneider, Vice President Electric Operations
13.	Architect/Engineer	Utility
14.	Nuclear Steam Supply System	GE
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Senior Inspector	E. McCabe
19.	IE Principal Inspector	W. Baunack
20.	Licensing Project Manager	P. Polk
21.	Docket Number	50-220
22.	License Number/Date of Issue	DPR-63, 12/26/74
23.	Public Document Room	State University College of Oswego Penfield Library · Documents Oswego, NY 13126 (315)341-2323

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO TS 6.8.1 & FACILITY PROCEDURE N1-OP-12, A VALVE REQUIRED TO BE LOCKED WAS FOUND UNLOCKED.
(8001 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

MANAGERIAL ITEMS:

- Mr. T. Perkins has been promoted from Station Superintendent to General Superintendent Nuclear Generation. Mr. T. Roman, Unit 2 Operations Supervisor has been named Station Superintendent. These position changes became effective April 1.

PLANT STATUS:

- The unit operated continuously at essentially full power on four recirculation pumps. Representative stack and offgas release rates are 10 and 17,300 microcuries per second, respectively.

LAST IE SITE INSPECTION DATE: 3/3-7/80

INSPECTION REPORT NO: 50-220/80-02

SUMMARY

INSPECTION SUMMARY

- No inspections completed during this report period.

NRC POWER RESTRICTIONS: NONE

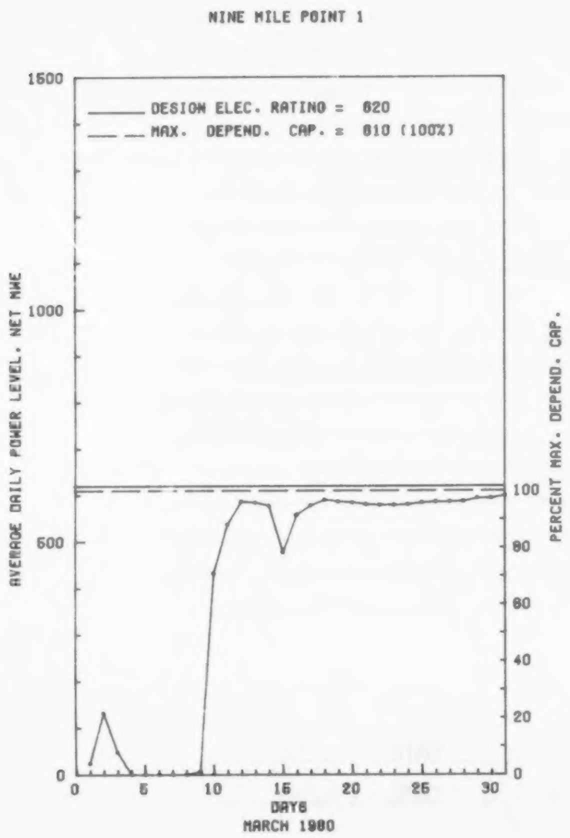
REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
80-007/03L	03/03/80	03/20/80	30 Day LER	Main steam isolation valves failed to close
80-008/03L	03/15/80	03/20/80	30 Day LER	Four hour loss of stack sample flow

REVIEWED BY OMPA LOEB: John J. DATE: 5-12-80

REVIEWED BY NRR DOR: DATE: 5/13/80

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-220
2. Utility Contact: B. Hawksley (315) 343-2110 x1312
3. Licensed Thermal Power (MWt): 1850
4. Nameplate Rating (Gross MWe): 755 x 0.85 = 642
5. Design Electrical Rating (Net MWe): 620
6. Maximum Dependable Capacity (Gross MWe): 630
7. Maximum Dependable Capacity (Net MWe): 610
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	2,184.0	91,296.0
12. Hours Reactor Was Critical	575.4	2,035.4	67,358.1
13. Reactor Reserve Shutdown Hours	.0	.0	1,204.2
14. Hours Generator On-Line	552.3	1,975.3	64,785.8
15. Unit Reserve Shutdown Hours	.0	.0	20.2
16. Gross Thermal Energy (MWH)	943,799	3,381,456	105,688,716
17. Gross Electrical Energy (MWH)	316,319	1,139,213	34,648,723
18. Net Electrical Energy (MWH)	306,061	1,101,554	33,551,319
19. Unit Service Factor	74.2	90.4	71.0
20. Unit Availability Factor	74.2	90.4	71.0
21. Unit Capacity Factor (MDC Net)	67.4	82.7	60.2
22. Unit Capacity Factor (DER Net)	66.4	81.4	59.3
23. Unit Forced Outage Rate	.0	.0	9.0

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-09	03/01/80	S	31.2	H	1				SHUTDOWN TO INSTALL RELIEF/SAFETY VALVE POSITION INDICATORS.
80-10	03/03/80	S	160.5	H	1				UNIT RECALLED BY POWER CONTROL THEN RELEASED TO START SCHEDULED OUTAGE.
80-11	03/14/80	S	.0	H	5				LOAD REDUCTION TO 60% POWER TO PULL FLUX SHAPING RODS.

SUMMARY

OPERATION WAS ROUTINE DURING THE PERIOD WITH THE NO. 15 REACTOR RECIRCULATION PUMP OUT OF SERVICE DUE TO MECHANICAL PROBLEMS. THE UNIT WAS SHUTDOWN ON 03/01/80 TO INSTALL NRC-REQUIRED RELIEF VALVE/SAFETY VALVE POSITION INDICATORS, BUT WAS RECALLED TO SERVICE BECAUSE OF A SYSTEM GENERATION SHORTAGE. THE INDICATORS WERE INSTALLED, AND OTHER MAINTENANCE WAS COMPLETED, BETWEEN 03/03/80 AND 03/09/80. LOAD WAS REDUCED ON 03/14/80 FOR CONTROL ROD FLUX SHAPING.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

NORTH ANNA 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Louisa County, Virginia 40 Miles NW of Richmond, Virginia
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	PWR
4.	Licensed Power Level (MWT)	2775
5.	Design Electrical Rating (MWE Net)	907
6.	Date of Initial Criticality	April 5, 1978
7.	Date of Commercial Operation	June 6, 1978
8.	Condenser Cooling Method	Once through
9.	Condenser Cooling Water Source	Lake Anna

Utility & Contractor Information

10.	Licensee	Virginia Electric & Power Co.
11.	Corporate Address	P. O. Box 26666 Richmond, Virginia 23261
12.	Corporate Contact	J. H. Ferguson Exec. V.P. - Power
13.	Architect/Engineer	Stone & Webster
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. Lewis
18.	IE Senior Inspector	P. J. Kellog
19.	IE Resident Inspector	M. S. Kidd (703) 894-5827
20.	Licensing Project Manager	L. Engle
21.	Docket Number	50-338
22.	License Number/Date of Issue	NPF-4, 11/26/77
23.	Public Document Room	Alderman Library/Manuscripts Dept. Univ. of Virginia/Charlottesville VA 22901 & Louisa County Courthouse, Louisa, VA 23093

INSPECTION STATUS

ENFORCEMENT STATUS

CARBON DIOXIDE FIRE PROTECTION SYSTEM FOR THE FUEL OIL PUMP HOUSE WAS NOT INSPECTED AT 6 MONTH INTERVALS DURING 1979 AS REQUIRED BY TECHNICAL SPECIFICATIONS SECTION 4.7.14.3.
(8003 INFRACTION)

A FIRE BRIGADE DRILL WAS NOT CONDUCTED FOR THE FIRE BRIGADES DURING THE LAST QUARTER OF 1979. TECHNICAL SPECIFICATIONS REQUIRE A DRILL FOR EACH BRIGADE AT LEAST ONCE EVERY 92 DAYS.
(8003 INFRACTION)

THE HOSE RACK FIRE PROTECTION SYSTEMS WERE NOT INSPECTED WITHIN 31 DAYS FOLLOWING THE MARCH 8, 1979 AND SEPTEMBER 7, 1978 INSPECTIONS AS REQUIRED BY TECHNICAL SPECIFICATIONS SECTION 4.7.14.5.
(8003 INFRACTION)

SECTION IV.E.3.B OF THE FIRE PROTECTION SYSTEMS REVIEW STATES THAT ALL FIRE PROTECTION SYSTEM CONTROL VALVES WILL EITHER BE ELECTRICALLY SUPERVISED OR SEALED IN THE CORRECT POSITION AND INSPECTED WEEKLY. A NUMBER OF THE FIRE PROTECTION CONTROL VALVES ARE INSPECTED MONTHLY IN LIEU OF THE SPECIFIED WEEKLY INSPECTIONS.
(8003 DEVIATION)

SECTION IV.B.3.C OF THE LICENSEE'S FIRE PROTECTION SYSTEMS REVIEW STATES THAT ONLY FIRE RETARDANT TREATED WOOD IS ALLOWED IN SAFETY RELATED AREAS OF THE PLANT. HOWEVER, COMBUSTIBLE WOOD SCAFFOLDING WAS NOTED IN SEVERAL SAFETY RELATED AREAS WITHIN THE PLANT.
(8003 DEVIATION)

THE PLANT'S ADMINISTRATIVE FIRE PROTECTION PROCEDURES DO NOT FULLY MEET THE NRC GUIDELINES OF THE NRC DOCUMENT "NUCLEAR PLANT FIRE PROTECTION FUNCTIONAL RESPONSIBILITIES, ADMINISTRATIVE CONTROLS AND QUALITY ASSURANCE" AS COMMITTED BY RESPONSE TO QUESTION NO. 31 IN SUPPLEMENT NO. 3 OF THE LICENSEE'S FIRE PROTECTION SYSTEM REVIEW.
(8003 DEVIATION)

CONTRARY TO THE REQUIREMENTS OF TECHNICAL SPECIFICATION 6.10.1.D, RECORDS OF SURVEILLANCE TESTS, 1-PT-11, PERFORMED TO MEET THE REQUIREMENTS OF TECHNICAL SPECIFICATION 4.1.1.1.2 WERE NOT ADEQUATE TO CONFIRM SATISFACTORY RESULTS.
(8005 DEFICIENCY)

AS REQUIRED BY CRITERION III OF APPENDIX B TO 10 CFR 50, "DESIGN CONTROL" AND SECTION 17.2.3 OF VIRGINIA ELECTRIC AND POWER COMPANY'S "TOPICAL REPORT - QUALITY ASSURANCE PROGRAM - OPERATIONS PHASE" (VEP-3-A) DESIGN CONTROL MEASURES SHALL ASSURE DELINEATION OF ACCEPTANCE CRITERIA FOR INSPECTIONS AND TESTS. INSTRUCTIONS FOR FINAL DESIGN IMPLEMENTING AND TEST PROCEDURES ON PAGE 37 OF SECTION 3 OF THE NUCLEAR POWER STATION QUALITY ASSURANCE MANUAL REQUIRE THAT ACCEPTANCE CRITERIA BE INCLUDED IN SUCH PROCEDURES. UNIT 1 DESIGN CHANGE 78-570, REPLACEMENT OF LIMIT SWITCHES ON INSIDE CONTAINMENT ISOLATION VALVES", DID NOT DELINEATE ACCEPTANCE CRITERIA FOR VALVE STROKE TIMES MEASURED AFTER PERFORMANCE OF THE DESIGN CHANGE.
(8007 INFRACTION)

TECHNICAL SPECIFICATION 6.8.1.E REQUIRES THAT WRITTEN PROCEDURES BE IMPLEMENTED WHICH COVER, IN PART, EMERGENCY PLAN IMPLEMENTATION. APPENDIX 8.5 OF THE EMERGENCY PLAN DESCRIBES EQUIPMENT TO BE MAINTAINED IN EMERGENCY HEALTH PHYSICS KITS. PROCEDURE HP-ADM-EP-1 REQUIRES AN INVENTORY OF THREE EMERGENCY HEALTH PHYSICS KITS MONTHLY (30 + 3 DAYS). CONTRARY TO THE ABOVE, THE THREE EMERGENCY HEALTH PHYSICS KITS WERE NOT INVENTORIED DURING THE MONTH OF NOVEMBER 1979, WITHIN THE PRESCRIBED TIME PERIOD.
(8009 DEFICIENCY)

TECHNICAL SPECIFICATION 6.2.2.F (ADMENDMENT 3, EFFECTIVE MAY 1, 1978) REQUIRED AT LEAST FIVE FIRE BRIGADE MEMBERS ON SITE AT ALL TIMES, IN ADDITION TO THE MINIMUM SHIFT CREW REQUIRED FOR A SAFE, ORDERLY SHUTDOWN OF THE PLANT. CONTRARY TO THE ABOVE, THE DAY SHIFT ON OCTOBER 28, 1978, CONSISTED OF ONLY FOUR FIRE BRIGADE MEMBERS. THIS WAS PREVIOUSLY IDENTIFIED AS AN UNRESOLVED ITEM AND IS APPLICABLE TO UNIT 1.
(8009 INFRACTION)

SEE CONTINUATION PAGE 0-9A

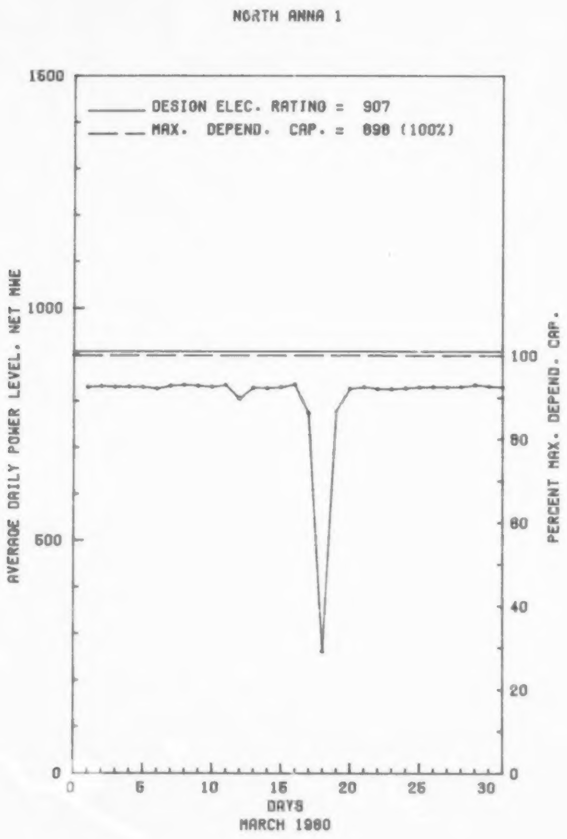
REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-156/ 03L-0	12/30/79	01/29/80	30 DAY LER	REACTOR COOLANT SYSTEM SNUBBER FOUND INOPERABLE.
80-003/ 03L-0	01/21/80	02/19/80	30 DAY LER	POWER RANGE CHANNEL N44 INOPERABLE.
80-004/ 03L-0	01/09/80	02/08/80	30 DAY LER	THE INNER EQUIPMENT HATCH EMERGENCY ESCAPE LOCK DOOR SEAL WAS DISCOVERED LEAKING.
80-005/ 03L-0	01/10/80	02/05/80	30 DAY LER	VALVE TV-MS101C WOULD NOT FUNCTION DURING OPERATION.
80-006/ 03L-0	01/11/80	02/07/80	30 DAY LER	CASING COOLING VALVE MOV-RS-105A DISCOVERED INOPERABLE.
80-007/ 03L-0	01/11/80	02/06/80	30 DAY LER	CONTAINMENT TEMPERATURE BELOW LIMITS.
80-009/ 03L-0	01/11/80	02/06/80	30 DAY LER	THREE SAFETY VALVES ON THE MAIN STEAM SYSTEM DID NOT LIFT AT PRESCRIBED SETPOINTS.
80-011/ 03L-0	01/15/80	02/13/80	30 DAY LER	B VENTILATION STACK RADIATION MONITORS INOPERABLE.
80-012/ 03L-0	01/11/80	02/07/80	30 DAY LER	VALVE MS101A FAILED TO OPERATE IN THE PRESCRIBED TIME DURING 1A ROUTINE SURVEILLANCE.
80-013/ 03L-0	01/14/80	02/11/80	30 DAY LER	STEAM DRIVEN AUXILIARY FEEDWATER PUMP INOPERABLE DURING MAINTENANCE.
80-014/ 03L-0	01/13/80	02/11/80	30 DAY LER	STEAM DRIVEN AUXILIARY FEEDWATER PUMP FOUND INOPERABLE.
80-015/ 03L-0	01/16/80	02/07/80	30 DAY LER	REACTOR COOLANT SYSTEM TEMPERATURE BELOW LIMITS WHILE THE REACTOR WAS CRITICAL DURING PHYSICS TESTING.
80-016/ 01T-0	01/16/80	01/28/80	14 DAY LER	PIPING HANGERS ON THE SERVICE WATER SCREEN WASH SYSTEM PIPING OVERSTRESSED DUE TO VALVE WEIGHT ERRORS.
80-017/ 01T-0	01/22/80	02/05/80	14 DAY LER	QUADRANT LOWER TILT BEING EXPERIENCED AT 5 PERCENT POWER.
80-019/ 01T-0	01/24/80	02/06/80	14 DAY LER	SERVICE WATER RADIATION MONITORING PUMPS NOT ENVIRONMENTALLY QUALIFIED.
80-020/ 03L-0	01/29/80	02/28/80	30 DAY LER	B CASING COOLING PUMP INOPERABLE WHILE CONDUCTING A DESIGN CHANGE.
80-021/ 03L-0	02/02/80	02/28/80	30 DAY LER	EMERGENCY DIESEL 1H TRIPPED DURING ROUTINE SURVEILLANCE.
80-022/ 03L-0	02/08/80	03/03/80	30 DAY LER	QUADRANT POWER TILT EXCEEDED LIMITS TWICE DURING ROUTINE SURVEILLANCE.
80-023/ 03L-0	02/08/80	03/10/80	30 DAY LER	CONTAINMENT HI-HI PRESSURE CHANNEL FAILED TO TRIP DURING ROUTINE SURVEILLANCE.
80-028/ 0TT-0	02/23/80	03/07/80	14 DAY LER	SAFETY INJECTION SYSTEM ACTUATION DUE TO TESTING.

REVIEWED BY OMPA LOEB: [illegible] **DATE:** /19/80
REVIEWED BY NRR DOR: B Engle **DATE:** 5/13/80

NORTH ANNA 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-338
2. Utility Contact: W. R. Madison (703) 894-5151
3. Licensed Thermal Power (MWt): 2775
4. Nameplate Rating (Gross MWe): 1089 x 0.9 = 980
5. Design Electrical Rating (Net MWe): 907
6. Maximum Dependable Capacity (Gross MWe): 928
7. Maximum Dependable Capacity (Net MWe): 898
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	2,184.0	15,961.0
12. Hours Reactor Was Critical	744.0	1,691.9	11,857.1
13. Reactor Reserve Shutdown Hours	.0	37.0	88.5
14. Hours Generator On-Line	744.0	1,542.7	11,595.8
15. Unit Reserve Shutdown Hours	.0	.0	0
16. Gross Thermal Energy (MWH)	2,016,657	3,706,771	29,986,684
17. Gross Electrical Energy (MWH)	636,429	1,160,508	9,506,857
18. Net Electrical Energy (MWH)	601,060	1,088,034	8,941,480
19. Unit Service Factor	100.0	70.6	72.7
20. Unit Availability Factor	100.0	70.6	72.7
21. Unit Capacity Factor (MDC Net)	90.0	55.5	62.4
22. Unit Capacity Factor (DER Net)	89.1	54.9	61.8
23. Unit Forced Outage Rate	.0	6.6	10.0

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-11	03/17/80	F	.0	A	5				SIGNIFICANT POWER REDUCTION OF >20% OF THE AVERAGE DAILY POWER LEVEL.

SUMMARY

OPERATION WAS ROUTINE DURING THE REPORT PERIOD, EXCEPT FOR A LOAD REDUCTION ON 03/17/80 DUE TO ARCING THE 500 KV BUS DISCONNECTS AND GENERATOR OUTPUT DISCONNECTS IN THE STATION SWITCHYARD. LOAD REMAINED REDUCED UNTIL 03/19/80 BECAUSE OF UNSTABLE STEAM GENERATOR CONDUCTIVITY AND AN OIL LEAK IN THE "A" FEED PUMP.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

best copy available

CONTINUATION PAGE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

UNIT IS OPERATING AT 100% POWER.

♦ LAST IE SITE INSPECTION DATE: MARCH 27, 1980

♦ INSPECTION REPORT NO: 50-338/80-13

SUMMARY

INSPECTION SUMMARY

♦ INSPECTION FEBRUARY 12-15 (80-03): THIS SPECIAL, UNANNOUNCED INSPECTION INVOLVED 25 INSPECTOR-HOURS ON SITE IN THE AREAS OF FIRE PROTECTION/PREVENTION. OF THE AREA INSPECTED, THREE ITEMS OF NONCOMPLIANCE WERE FOUND (INFRACTION - EXCESSIVE TIME BETWEEN INSPECTIONS OF CARBON DIOXIDE SYSTEM FOR FUEL OIL PUMP HOURS; INFRACTION - EXCESSIVE TIME BETWEEN INSPECTIONS OF FIRE HOSE STATIONS; AND, INFRACTION - INADEQUATE FIRE BRIGADE DRILLS FOR 1979) AND THREE DEVIATIONS WERE FOUND (INADEQUATE ADMINISTRATIVE FIRE PROTECTION PROCEDURES; COMBUSTIBLE WOOD USED IN SAFETY-RELATED AREAS; AND, SUBSTANDARD SUPERVISION OF FIRE PROTECTION CONTROL VALVES).

INSPECTION JANUARY 22-25 (80-04): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 18 INSPECTOR-HOURS ON SITE IN THE AREAS OF LICENSEE EVENT REPORTS, PREVIOUS INSPECTOR CONCERNS, AND LICENSE CONDITIONS. IT ALSO INCLUDED 5 INSPECTOR-HOURS AT VEPCO CORPORATE OFFICE IN RICHMOND, VIRGINIA IN THE AREA OF A RECENT REPORTABLE EVENT. OF THE FOUR AREAS INSPECTED, NO NEW ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION FEBRUARY 11-15 (80-05): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 31 INSPECTOR-HOURS ON SITE IN THE AREAS OF CORE PERFORMANCE MONITORING (UNIT 1), POST-REFUELING STARTUP TESTS (UNIT 1) AND INITIAL STARTUP TESTS (UNIT 2). OF THE THREE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN TWO AREAS; ONE ITEM OF NONCOMPLIANCE WAS FOUND IN ONE AREA (DEFICIENCY - FAILURE TO DOCUMENT SURVEILLANCE TESTS).

INSPECTION JANUARY 2-18, 24-25 AND JANUARY 31 - FEBRUARY 1 (80-07): THIS ROUTINE INSPECTION BY THE RESIDENT INSPECTOR INVOLVED THIRTY-EIGHT (38) HOURS ON SITE IN THE AREAS OF PREVIOUSLY IDENTIFIED ENFORCEMENT ITEMS, PREVIOUSLY IDENTIFIED OPEN ITEMS, LICENSEE EVENT REPORTS, ITEMS REQUIRING RESOLUTION PRIOR TO UNIT 1 RESTART AFTER REFUELING, AND PLANT TOURS. OF THE FIVE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN FOUR AREAS; ONE APPARENT ITEM OF NONCOMPLIANCE WAS FOUND IN ONE AREA (INFRACTION - FAILURE TO ESTABLISH TEST ACCEPTANCE CRITERIA).

INSPECTION FEBRUARY 19-22 (80-09): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 43 INSPECTOR-HOURS ON SITE IN THE AREAS OF COORDINATION WITH OFFSITE SUPPORT AGENCIES; EMERGENCY FACILITIES, EQUIPMENT AND PROCEDURES; EMERGENCY TRAINING; EMERGENCY PLANNING AUDITS; EMERGENCY DRILLS; FOLLOWUP ON PREVIOUSLY IDENTIFIED INSPECTION FINDINGS AND FOLLOWUP ON IE BULLETINS. A PREOPERATIONAL INSPECTION WAS PERFORMED ON FEBRUARY 20, 1980, TO DETERMINE THE STATUS OF COMPLETION OF UNIT 2'S AUXILIARY SHUTDOWN PANEL. OF THE SEVEN AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN FIVE AREAS; TWO ITEMS OF NONCOMPLIANCE WERE FOUND IN TWO AREAS (DEFICIENCY - FAILURE TO INVENTORY EMERGENCY KITS; INFRACTION - FAILURE TO MAINTAIN THE MINIMUM NUMBER OF FIRE BRIGADE MEMBERS ON EACH SHIFT).

INSPECTION FEBRUARY 19-22 (80-10): THIS SPECIAL, ANNOUNCED INSPECTION INVOLVED 56 INSPECTOR-HOURS ON SITE IN THE AREAS OF SMALL BREAK LOSS OF COOLANT PROCEDURES AND TRAINING, AND THE REACTOR TRIP OF FEBRUARY 20. OF THE THREE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION FEBRUARY 4-19 AND 25-29 (80-11): THIS ROUTINE INSPECTION BY THE RESIDENT INSPECTOR INVOLVED 12 HOURS ON SITE IN THE AREAS OF PREVIOUSLY IDENTIFIED OPEN ITEMS AND REPORTABLE PLANT EVENTS. WITHIN THE TWO AREAS INSPECTED, ONE APPARENT ITEM OF NONCOMPLIANCE WAS IDENTIFIED (INFRACTION - FAILURE TO PERFORM A SAFETY EVALUATION ON A PROPOSED TEST AS REQUIRED BY TECHNICAL SPECIFICATIONS).

OCONEE 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	South Carolina/Oconee 30 miles W of Greenville
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2568
5.	Design Electrical Rating (MWE Net)	887
6.	Date of Initial Criticality	April 19, 1973
7.	Date of Commercial Operation	July 15, 1973
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Keowee Lake

Utility & Contractor Information

10.	Licensee	Duke Power Company
11.	Corporate Address	422 S. Church Street Charlotte, North Carolina 28242
12.	Corporate Contact	W. O. Parker, Jr.
13.	Architect/Engineer	DPC and Bechtel (Containment Bldg.)
14.	Nuclear Steam Supply System	B&W
15.	Constructor	DPC

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R.C. Lewis
18.	IE Senior Inspector	
19.	IE Resident Inspector	F. Jape (803) 882-6927
20.	Licensing Project Manager	M. B. Fairtile
21.	Docket Number	50-269
22.	License Number/Date of Issue	DPR-38, 2/6/73
23.	Public Document Room	Oconee County Library 201 S. Spring Street Walhalla, South Carolina 29691

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO 10 CFR 71.5, ON DECEMBER 18, 1979, A RADIOACTIVE WASTE SHIPMENT (CASK) FROM YOUR OCONEE PLANT, UPON ARRIVAL AT CHEM-NUCLEAR'S WASTE DISPOSAL SITE AT BARNWELL, SOUTH CAROLINA WAS FOUND TO BE MARKED WITH "EMPTY" LABELS IN ADDITION TO "RADIOACTIVE LSA" LABELS. 49 CFR 172.502(A)(2) SPECIFIES THAT NO PERSON MAY AFFIX OR DISPLAY ON A MOTOR VEHICLE ANY PLACARD UNLESS THE PLACARD REPRESENTS A HAZARD OF THE HAZARDOUS MATERIAL BEING TRANSPORTED.
(7945 DEFICIENCY)

CONTRARY TO 10 CFR 71.5, ON FEBRUARY 6, 1980, A SHIPMENT OF 150 DRUMS OF RADIOACTIVE WASTE FROM YOUR OCONEE PLANT ARRIVED AT CHEM-NUCLEAR WASTE DISPOSAL SITE IN BARNWELL, SC NOT PROPERLY MARKED "RADIOACTIVE - LSA" AS REQUIRED BY 49 CFR 173.392(C)8.
(8011)

CONTRARY TO 10 CFR 71.5, ON FEBRUARY 6, 1980, A SHIPMENT OF 150 DRUMS CONTAINING RADIOACTIVE WASTE FROM YOUR OCONEE PLANT ARRIVED AT CHEM-NUCLEAR WASTE DISPOSAL SITE IN BARNWELL, SC NOT BRACED TO PREVENT SHIFTING OF LADING (DRUMS) AS REQUIRED BY 49 CFR 173.392(C)6.
(8011 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

- 100% POWER.
- LAST IE SITE INSPECTION DATE: MARCH 1-31, 1980
- INSPECTION REPORT NO: 50-269/80-13

SUMMARY

INSPECTION SUMMARY

- INSPECTION JANUARY 14-17 AND 22-24, (80-03): THIS ROUTINE UNANNOUNCED INSPECTION INVOLVED 21 INSPECTOR-HOURS ON SITE IN THE AREAS OF IE BULLETINS 79-13 AND 79-17, UNITS 1, 2 AND 3; AUXILIARY FEEDWATER NOZZLE THERMAL SLEEVE INSPECTION AND CRACK REPAIR, CRD STATOR MOTOR DROP DURING MAINTENANCE, RPV FLANGE O-RING REPAIR. OF THE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION JANUARY 23-30 (80-06): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 196 INSPECTOR-HOURS ON SITE IN THE AREA OF CONTAINMENT INTEGRATED LEAK RATE TESTING WHICH INCLUDED INSPECTION OF TEST PROCEDURES, INSTRUMENT CALIBRATIONS, DATA PROCESSING, VALVE ALIGNMENT AND PROBLEMS ENCOUNTERED DURING THE TEST. OF THE AREA INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION FEBRUARY 25-29 (80-08): INCLUDED REVIEW OF THE LICENSEE'S SECURITY PLAN, SECURITY PROGRAM AUDIT, SECURITY ORGANIZATION, VITAL AND PROTECTED AREA PHYSICAL BARRIERS, LIGHTING, ACCESS CONTROLS, ALARM STATION, DETECTION AND ASSESSMENT AIDS, COMMUNICATIONS, TESTING AND MAINTENANCE, COMPENSATORY MEASURES, RESPONSE, KEYS/LOCKS, AND RECORDS/REPORTS. THE INSPECTION INVOLVED 26 INSPECTOR-HOURS ON SITE BY TWO NRC INSPECTORS. THE INSPECTION WAS BEGUN DURING AN OFFSHIFT PERIOD; 6 INSPECTION HOURS WERE ACCOMPLISHED DURING OFFSHIFT PERIOD. THE LICENSEE WAS FOUND TO BE IN COMPLIANCE WITH NRC REQUIREMENTS WITHIN THE AREAS EXAMINED DURING THE INSPECTION. THE CORRECTIVE ACTION FOR A PREVIOUSLY IDENTIFIED ITEM OF NONCOMPLIANCE (79-24-01) PERTAINING TO A VITAL AREA PHYSICAL BARRIER WAS NOT COMPLETED AND REMAINS OPEN.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

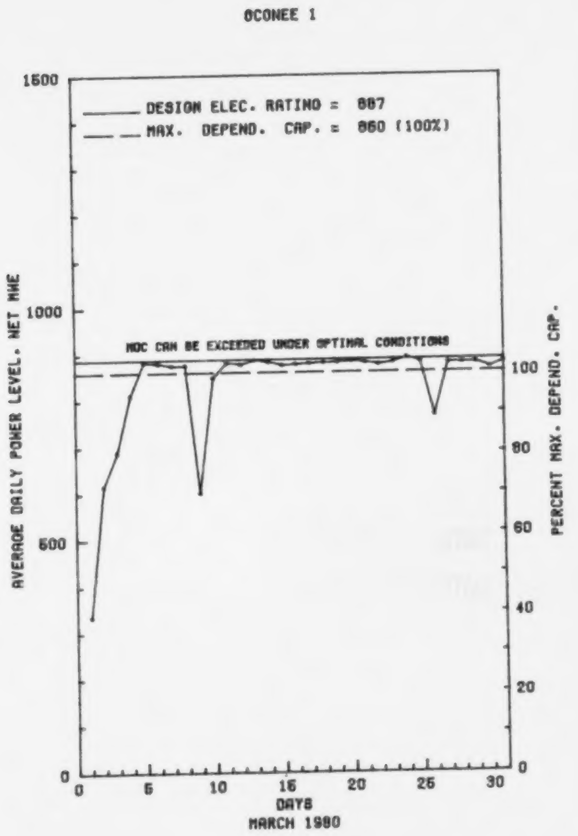
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
NONE				

REVIEWED BY OMPA LOEB: Bobe **DATE:** 5/12/80

REVIEWED BY NRR DOR: M B Fairtile **DATE:** 5/12/80

OCONEE 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-269
2. Utility Contact: J. A. Reavis (704) 373-8552
3. Licensed Thermal Power (MWt): 2568
4. Nameplate Rating (Gross MWe): 1038 x 0.9 = 934
5. Design Electrical Rating (Net MWe): 887
6. Maximum Dependable Capacity (Gross MWe): 899
7. Maximum Dependable Capacity (Net MWe): 860
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

CAPACITY FACTORS CALCULATED WITH A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	2,184.0	58,825.0
12. Hours Reactor Was Critical	744.0	858.5	41,373.2
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	744.0	806.8	38,753.9
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,813,478	1,868,148	91,086,267
17. Gross Electrical Energy (MWH)	647,050	666,870	31,581,170
18. Net Electrical Energy (MWH)	618,561	625,522	29,857,021
19. Unit Service Factor	100.0	36.9	65.9
20. Unit Availability Factor	100.0	36.9	65.9
21. Unit Capacity Factor (MDC Net)	96.7	33.3	68.8
22. Unit Capacity Factor (DER Net)	93.7	32.3	67.3
23. Unit Forced Outage Rate	.0	27.8	18.2

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
2	03/01/80	S	.0	B	5		ZZ	ZZZZZZ	POWER ESCALATION TESTING CONTINUES AT 40, 75, AND 90% POWER LEVELS. NEAR RATED POWER WAS REACHED ON 3/4.
3	03/09/80	F	.0	A	5		CH	TURBIN	FWPT "B" WAS REMOVED FROM SERVICE TO CHECK OIL PRESSURE PROBLEM.
4	03/26/80	F	.0	A	5		RB	CRDRVE	BLOWN FUSE IN "C" PHASE CAUSED RD. 7 IN GP. 7 TO DROP. POWER WAS REDUCED TO 55%. AFTER REPLACING FUSE, POWER INCREASE BEGAN. NEAR RATED POWER WAS REACHED ON 3/27.

SUMMARY

OPERATION WAS AT NEAR FULL POWER DURING THE REPORT PERIOD, FOLLOWING COMPLETION OF POWER ESCALATION TESTING ON 03/04/80. LOAD REDUCTIONS OCCURRED ON 03/09/80 FOR TESTING OF A FEEDWATER PUMP TURBINE OIL PUMP, AND ON 03/26/80 WHEN A BLOWN FUSE IN THE "C" PHASE CAUSED A CONTROL ELEMENT ASSEMBLY TO DROP.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examinstion
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

OCONEE 2

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	South Carolina/Oconee 30 miles W of Greenville
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2568
5.	Design Electrical Rating (MWE Net)	887
6.	Date of Initial Criticality	November 11, 1973
7.	Date of Commercial Operation	September 9, 1974
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Keowee Lake

Utility & Contractor Information

10.	Licensee	Duke Power Company
11.	Corporate Address	422 S. Church Street Charlotte, North Carolina 28242
12.	Corporate Contact	W. O. Parker, Jr.
13.	Architect/Engineer	DPC/Bechtel
14.	Nuclear Steam Supply System	B&W
15.	Constructor	DPC

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R.C. Lewis
18.	IE Senior Inspector	
19.	IE Resident Inspector	F. Jape (803) 882-6927
20.	Licensing Project Manager	M. B. Fairtile
21.	Docket Number	50-270
22.	License Number/Date of Issue	DPR-47, 10/6/73
23.	Public Document Room	Oconee County Library 201 S. Spring Street Walhalla, South Carolina 29691

INSPECTION STATUS

ENFORCEMENT STATUS

PIPING TO AUXILIARY BUILDING.
(7920 DEVIATION)

QA RECORDS STORAGE.
(7920 DEVIATION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

+ NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

REFUELING OUTAGE.

+ LAST IE SITE INSPECTION DATE: MARCH 1-31, 1980

+ INSPECTION REPORT NO: 50-270/80-08

SUMMARY

INSPECTION SUMMARY

+ INSPECTION JANUARY 14-17 AND 22-24 (80-02): THIS ROUTINE UNANNOUNCED INSPECTION INVOLVED 21 INSPECTOR-HOURS ON SITE IN THE AREAS OF IE BULLETINS 79-13 AND 79-17, UNITS 1, 2 AND 3; AUXILIARY FEEDWATER NOZZLE THERMAL SLEEVE INSPECTION AND CRACK REPAIR, CRD STATOR MOTOR DROP DURING MAINTENANCE, RPV FLANGE O-RING REPAIR. OF THE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION FEBRUARY 25-29 (80-08): INCLUDING REVIEW OF THE LICENSEE'S SECURITY PLAN, SECURITY PROGRAM AUDIT, SECURITY ORGANIZATION, VITAL AND PROTECTED AREA PHYSICAL BARRIERS, LIGHTING, ACCESS CONTROLS, ALARM STATION, DETECTION AND ASSESSMENT AIDS, COMMUNICATIONS, TESTING AND MAINTENANCE, COMPENSATORY MEASURES, RESPONSE, KEYS/LOCKS, AND RECORDS/REPORTS. THE INSPECTION INVOLVED 27 INSPECTOR-HOURS ON SITE BY TWO NRC INSPECTORS. THE INSPECTION WAS BEGUN DURING AN OFFSHIFT PERIOD; 7 INSPECTION HOURS WERE ACCOMPLISHED DURING OFFSHIFT PERIODS. THE LICENSEE WAS FOUND TO BE IN COMPLIANCE WITH NRC REQUIREMENTS WITHIN THE AREAS EXAMINED DURING THE INSPECTION. THE CORRECTIVE ACTION FOR A PREVIOUSLY IDENTIFIED ITEM OF NONCOMPLIANCE (79-24-01) PERTAINING TO A VITAL AREA PHYSICAL BARRIER WAS NOT COMPLETED AND REMAINS OPEN.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

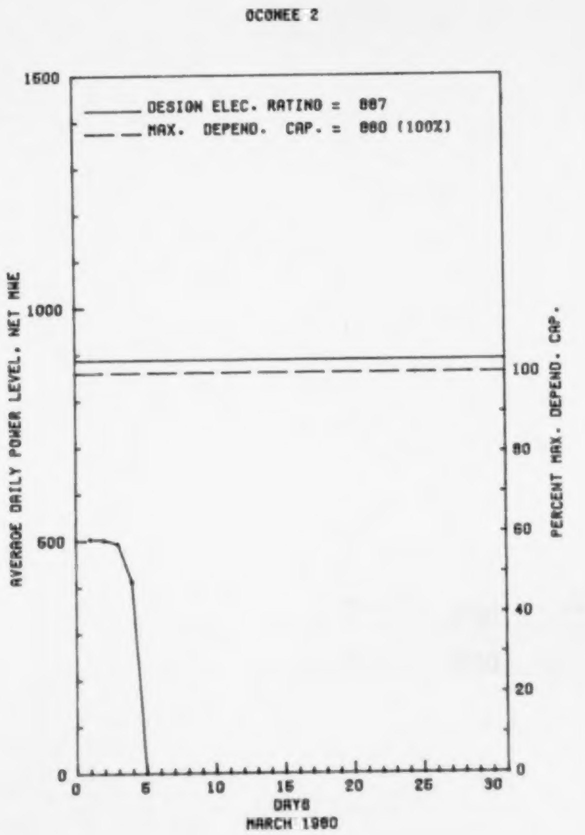
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
80-01/ 01T	02/20/80	03/05/80	14 DAY LER	CORE FLOOD TANK 2A PRESSURE EXCEEDED TECHNICAL SPECIFICATION LIMIT 625 PSI.

REVIEWED BY OMPA LOEB: PEBobe **DATE:** 5/12/80

REVIEWED BY NRR DOR: M. B. Fairtile **DATE:** 5/12/80

OCONEE 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-270
2. Utility Contact: J. A. Reavis (704) 373-8552
3. Licensed Thermal Power (MWt): 2568
4. Nameplate Rating (Gross MWe): 1038 x 0.9 = 934
5. Design Electrical Rating (Net MWe): 887
6. Maximum Dependable Capacity (Gross MWe): 899
7. Maximum Dependable Capacity (Net MWe): 860
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

CAPACITY FACTORS CALCULATED WITH A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	2,184.0	48,745.0
12. Hours Reactor Was Critical	97.1	1,536.3	35,132.3
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	94.1	1,521.8	34,297.9
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	144,313	3,326,000	81,373,405
17. Gross Electrical Energy (MWH)	49,020	1,126,140	27,640,496
18. Net Electrical Energy (MWH)	43,221	1,069,186	26,232,944
19. Unit Service Factor	12.6	69.7	70.4
20. Unit Availability Factor	12.6	69.7	70.4
21. Unit Capacity Factor (MDC Net)	6.8	56.9	62.3
22. Unit Capacity Factor (DER Net)	6.5	55.2	60.7
23. Unit Forced Outage Rate	.0	.8	19.3

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 08/10/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
3C	03/01/80	S	.0	H	5		RC	FUELXX	POWER REDUCTION DUE TO FUEL DEPLETION.
3D	03/04/80	S	649.9	C	1		RC	FUELXX	SCHEDULED REFUELING AND PIPE HANGER/SUPPORT INSPECTION AND MODIFICATION.

SUMMARY

THE UNIT WAS SHUTDOWN ON 03/04/80 FOR REFUELING AND PIPE HANGER/SUPPORT INSPECTION AND MODIFICATION.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

OCONEE 3

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	South Carolina/Oconee 30 miles W of Greenville
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2568
5.	Design Electrical Rating (MWE Net)	887
6.	Date of Initial Criticality	September 5, 1974
7.	Date of Commercial Operation	December 16, 1974
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Keowee Lake

Utility & Contractor Information

10.	Licensee	Duke Power Company
11.	Corporate Address	422 S. Church Street Charlotte, North Carolina 28242
12.	Corporate Contact	W. O. Parker, Jr.
13.	Architect/Engineer	DPC/Bechtel
14.	Nuclear Steam Supply System	B&W
15.	Constructor	DPC

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R.C. Lewis
18.	IE Senior Inspector	
19.	IE Resident Inspector	F. Jape (803) 882-6927
20.	Licensing Project Manager	M. B. Fairtile
21.	Docket Number	50-287
22.	License Number/Date of Issue	DPR-55, 7/19/74
23.	Public Document Room	Oconee County Library 201 S. Spring Street Walhalla, South Carolina 29691

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

I N S P E C T I O N   S T A T U S

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

+ NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

+ SHUTDOWN TO INSTALL MODIFICATIONS REQUIRED BY NUREG 0578.

+ LAST IE SITE INSPECTION DATE: MARCH 1-31, 1980

+ INSPECTION REPORT NO: 50-287/80-08

SUMMARY

INSPECTION SUMMARY

+ INSPECTION JANUARY 14-17 AND 22-24 (80-02): THIS ROUTINE UNANNOUNCED INSPECTION INVOLVED 21.5 INSPECTOR-HOURS ON SITE IN THE AREAS OF IE BULLETINS 79-13 AND 79-17, UNITS 1, 2 AND 3; AUXILIARY FEEDWATER NOZZLE THERMAL SLEEVE INSPECTION AND CRACK REPAIR, CRD STATOR MOTOR DROP DURING MAINTENANCE, RPV FLANGE O-RING REPAIR. OF THE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION FEBRUARY 25-29 (80-06): INCLUDED REVIEW OF THE LICENSEE'S SECURITY PLAN, SECURITY PROGRAM AUDIT, SECURITY ORGANIZATION, VITAL AND PROTECTED AREA PHYSICAL BARRIERS, LIGHTING, ACCESS CONTROLS, ALARM STATION, DETECTION AND ASSESSMENT AIDS, COMMUNICATIONS, TESTING AND MAINTENANCE, COMPENSATORY MEASURES, RESPONSE, KEYS/LOCKS, AND RECORDS/REPORTS. THE INSPECTION INVOLVED 27 INSPECTOR-HOURS ON SITE BY TWO NRC INSPECTORS. THE INSPECTION WAS BEGUN DURING AN OFFSHIFT PERIOD; 7 INSPECTION HOURS WERE ACCOMPLISHED DURING OFFSHIFT PERIODS. THE LICENSEE WAS FOUND TO BE IN COMPLIANCE WITH NRC REQUIREMENTS WITHIN THE AREAS EXAMINED DURING THE INSPECTION. THE CORRECTIVE ACTION FOR A PREVIOUSLY IDENTIFIED ITEM OF NONCOMPLIANCE (79-24-01) PERTAINING TO A VITAL AREA PHYSICAL BARRIER WAS NOT COMPLETED AND REMAINS OPEN.

REPORTS RECEIVED FROM LICENSEE

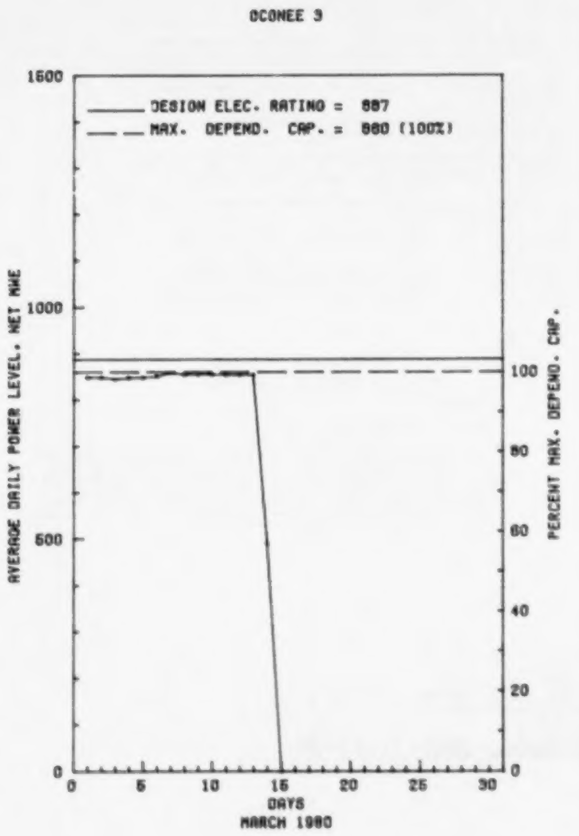
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
80-01/ 03L-0	02/01/80	02/29/80	30 DAY LER	BWST LEVEL CHANNEL 2 FAILED HIGH WHEN LEVEL TRANSMITTER FROZE DUE TO LOSS OF STRIP HEATERS.
80-02/ 03L-0	02/01/80	02/29/80	30 DAY LER	STARTUP BREAKER FOR MAIN FEEDER BUS 2 FAILED TO CLOSE DURING OPERABILITY TEST DUE TO FAULTY RESISTOR IN CLOSING COIL MONITOR RELAY CIRCUIT.
80-03/ 03L-0	02/05/80	03/06/80	30 DAY LER	MAIN FEEDER BUS 1 ISOLATED WHEN SMOKING BUS CONNECTION DISCOVERED. MAIN FEEDER BUSES WERE FED BY TRANSFORMER IN PARALLEL. SECONDARY VOLTAGE MISMATCH OVERLOADED ONE.

REVIEWED BY OMPA LOEB: P E Bobe **DATE:** 5/12/80

REVIEWED BY NRR DOR: M. B. Fairtile **DATE:** 5/12/80

OCONEE 3

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-287
2. Utility Contact: J. A. Reavis (704) 373-8552
3. Licensed Thermal Power (MWt): 2568
4. Nameplate Rating (Gross MWe): 1038 x 0.9 = 934
5. Design Electrical Rating (Net MWe): 887
6. Maximum Dependable Capacity (Gross MWe): 899
7. Maximum Dependable Capacity (Net MWe): 860
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

CAPACITY FACTORS CALCULATED WITH A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	2,184.0	46,392.0
12. Hours Reactor Was Critical	326.1	1,766.1	33,460.0
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	326.1	1,766.1	32,828.0
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	835,889	4,482,218	78,873,824
17. Gross Electrical Energy (MWH)	289,710	1,550,610	27,301,874
18. Net Electrical Energy (MWH)	275,261	1,483,952	25,983,508
19. Unit Service Factor	43.8	80.9	70.8
20. Unit Availability Factor	43.8	80.9	70.8
21. Unit Capacity Factor (MDC Net)	43.0	79.0	64.8
22. Unit Capacity Factor (DER Net)	41.7	76.6	63.2
23. Unit Forced Outage Rate	2.9	.6	17.3

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 04/21/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
5	03/14/80	F	9.9	A	3		HA	TURBIN	REACTOR TRIPPED DUE TO A SPURIOUS TURBINE/GENERATOR TRIP.
5A	03/15/80	S	408.0	D	9		ZZ	ZZZZZZ	NRC REQUIRED MODIFICATIONS NUREG-0578 AND OTHER NSM'S.

SUMMARY

OPERATION WAS ROUTINE UNTIL THE REACTOR TRIPPED ON 03/14/80 FOLLOWING A SPURIOUS TURBINE/GENERATOR TRIP. THE UNIT REMAINED SHUTDOWN THROUGH THE END OF THE PERIOD TO MAKE NRC-REQUIRED MODIFICATIONS.

LEGEND

[1] Type:
F–Forced
S–Scheduled

[2] Reason:
A–Equipment Failure
B–Maintenance or Test
C–Refueling
D–Regulatory Restriction
E–Operator Training & License Examination
F–Administrative
G–Operational Error
H–Other

[3] Method:
1–Manual
2–Manual Scram
3–Automatic Scram
4–Continuations
5–Load Reduction
9–Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

OYSTER CREEK

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	New Jersey/Ocean 9 miles S of Toms River
2.	Electric Reliability Council	Mid-Atlantic Area Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	1930
5.	Design Electrical Rating (MWE Net)	650
6.	Date of Initial Criticality	May 3, 1969
7.	Date of Commercial Operation	December 1969
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Barnegat Bay

Utility & Contractor Information

10.	Licensee	Jersey Central Power & Light Co.
11.	Corporate Address	Madison Avenue At Punch Bowl Road Morristown, New Jersey 07960
12.	Corporate Contact	I. Finfrock, Vice President Power Generation
13.	Architect/Engineer	Burns & Roe
14.	Nuclear Steam Supply System	GE
15.	Constructor	J. A. Jones

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	IE Senior Inspector	E. McCabe
19.	IE Principal Inspector	L. Briggs
20.	Licensing Project Manager	W. A. Paulson
21.	Docket Number	50-219
22.	License Number/Date of Issue	DPR-16, 4/9/69
23.	Public Document Room	Ocean County Library 15 Hooper Avenue Toms River, New Jersey 08753

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO 10 CFR 20.201(B), THE LICENSEE FAILED TO MAKE SURVEYS OF THE EFFLUENTS RELEASED FROM THE NEW RAD WASTE BUILDING.
(7918 INFRACTION)

CONTRARY TO 10 CFR 50.54(P), THE LICENSEE DID NOT NOTIFY THE DIRECTOR OF NUCLEAR REACTOR REGULATION OR THE APPROPRIATE REGIONAL OFFICE OF A CHANGE TO THE SECURITY PLAN, WHICH DID NOT DECREASE THE SAFEGUARDS EFFECTIVENESS OF THE PLAN, WITHIN THE REQUIRED 2 MONTHS AFTER THE CHANGE WAS MADE.
(7918 DEFICIENCY)

CONTRARY TO 10 CFR 50, APPENDIX B, CRITERION V & THE OQA PLAN, THE LICENSEE FAILED TO FOLLOW PROCEDURES IN THAT 3 FIRE DOORS WERE FOUND BLOCKED OPEN & WOOD UNCERTIFIED AS "FIRE RETARDANT" WAS FOUND IN THE FUEL STORAGE AREA.
(7918 INFRACTION)

CONTRARY TO 10 CFR 50, APPENDIX B, CRITERION V & THE OQA PLAN, THE LICENSEE FAILED TO REINSPECT & STORE WELD RODS AS REQUIRED BY PROCEDURE 3005.
(7918 DEFICIENCY)

CONTRARY TO 10 CFR 50, APPENDIX B, CRITERION V & THE OQA PLAN, SECTION X, THE LICENSEE FAILED TO FOLLOW PROCEDURE 2002 IN THAT NONCONFORMANCE/CORRECTIVE ACTION REPORTS WERE NOT RESPONDED TO WITHIN 10 DAYS & CLOSEOUT WAS NOT TIMELY.
(7918 INFRACTION)

CONTRARY TO 10 CFR 50, APPENDIX B, CRITERION V & THE OQA PLAN, SECTION X, THE LICENSEE FAILED TO IMPLEMENT A TRAINING PROGRAM AS SPECIFIED IN PROCEDURES 5002, 6002, 7002 & THE ADMINISTRATIVE TRAINING MANUAL, SECTION IV & A FORMAL TRAINING PROGRAM FOR HP TECHNICIANS WAS NOT ESTABLISHED.
(7918 INFRACTION)

CONTRARY TO 10 CFR 50, APPENDIX B, CRITERION V & THE OQA PLAN, SECTION X, INADEQUATE INSTALLATION INSTRUCTIONS EXISTED IN THAT INSTALLATION OF ANCHOR BOLTS & REPAIRS OF ABANDONED DRILL HOLES WERE NOT ACCOMPLISHED AS REQUIRED BY PROCEDURE 6009 & A JOB ORDER HAD NOT BEEN COMPLETED AS REQUIRED.
(7918 INFRACTION)

CONTRARY TO 10 CFR 50, APPENDIX B, CRITERION V & THE OQA PLAN, THE LICENSEE FAILED TO FOLLOW PROCEDURE 105 IN THAT OPERATING PROCEDURES & "AS-BUILT" DRAWINGS WERE NOT UPDATED AFTER COMPLETION OF A MODIFICATION. ALSO WORK WAS PERFORMED ON Q-EQUIPMENT W/O WRITTEN PROCEDURES.
(7918 INFRACTION)

CONTRARY TO 10 CFR 50, APPENDIX B, CRITERION V & THE OQA PLAN, SECTION X, INSTALLATION DRAWINGS LACKED DETAILS REGARDING PIPE SUPPORTS & LACKED THE USE OF "SECOND NUT" REQUIREMENTS FOR PIPE SUPPORT U CLAMPS.
(7918 INFRACTION)

CONTRARY TO TS 4.5.K.1.A(2), A SAMPLE FROM THE STANDBY GAS TREATMENT SYSTEM CHARCOAL ABSORBERS WAS NOT TAKEN OR ANALYZED AS OF 11/2/79. THIS SURVEILLANCE IS REQUIRED EVERY 18 MONTHS.
(7918 INFRACTION)

CONTRARY TO TS 6.5.3.5.B, THE TRAINING & QUALIFICATIONS OF THE ENTIRE FACILITY STAFF WAS NOT AUDITED AT LEAST ONCE A YEAR.
(7918 INFRACTION)

CONTRARY TO TS 6.10.3, THE LICENSEE FAILED TO MAINTAIN A DUPLICATE FILE SYSTEM AT THE SITE & CORPORATE OFFICE.
(7918 INFRACTION)

CONTRARY TO TS 6.8.1, CONTROL OF EQUIPMENT PROCEDURE 108 DID NOT PROVIDE FOR INDEPENDENT VERIFICATION FOR PLACING/REMOVING TAGS, JUMPERS & LIFTED LEADS.
(7918 INFRACTION)

CONTRARY TO TS 6.8.1, WRITTEN PROCEDURES HAD NOT BEEN ESTABLISHED IN ORDER TO CALIBRATE VARIOUS RADIATION EFFLUENT & GASEOUS MONITORS.
(7918 INFRACTION)

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
80-004/ 03L	01/25/80	02/25/80	30 Day LER	Several leaks discovered in underground aluminum condensate lines. Leakage was due to galvanic corrosion
80-005/ 03L	02/05/80	02/28/80	30 Day LER	Reactor building ventilation monitor trip setpoints found high er than T.S. limits
80-006/ 03L	01/31/80	02/29/80	30 Day LER	Recirculation flow sensors at zero percent found out of tolerance on 6 of 8 channesl. Reactor scram for 3 of 8 above specified value due to flow snesor error
80-007/ 03L	02/13/80	03/06/80	30 Day LER	Low flow on SBGTS No. 1 due to slipping belts on exhaust fan. Belst replaced and system tested satisfactorily.
80-008/ 01T	02/15/80	02/29/80	24 Hours	Nine pipe clamps which connect snubbers to isolation condenser piping were found not installed per design. Identified under IEB 79-14.
80-009/ 03L	02/19/80	03/19/80	30 Day LER	Tube leakage on all containment spray heat exchangers. Tubes being replaced during refueling outage
80-010/ 01T	02/20/80	03/06/80	24 Hours	During as built verification 3 pipe hangers in the liquid poison system were identified as not installed per design. One add restraint in RWCU system also ident as incor installed.
80-011/ 03L	02/19/80	03/17/80	30 Day LER	Trip of SBGTS No. 1 when flow indicated zero because of a leaking instrument sensing line
80-013/ 01T	03/09/80	03/24/80	24 Hours	Fire system taken out of service to repair a leaking valve in the supply header
80-014/ 01T	03/02/80	03/24/80	24 Hours	Diesel Generator No. 1 failed to synchronize and tripped while undergoing surveillance testing during cold shutdown conditions

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

- A total of 28 cracks and/or crack indications were identified in the Core Spray System I and II spargers during inspection and evaluation of previously identified and reported cracks. An NRC/Licensee meeting was held on April 1, 1980 to discuss planned repairs.

PLANT STATUS:

- The plant is shutdown for refueling with return to operation scheduled for May 22, 1980.

- LAST IE SITE INSPECTION DATE: 3/28/80
- INSPECTION REPORT NO: 50-219/80-12

SUMMARY

INSPECTION SUMMARY

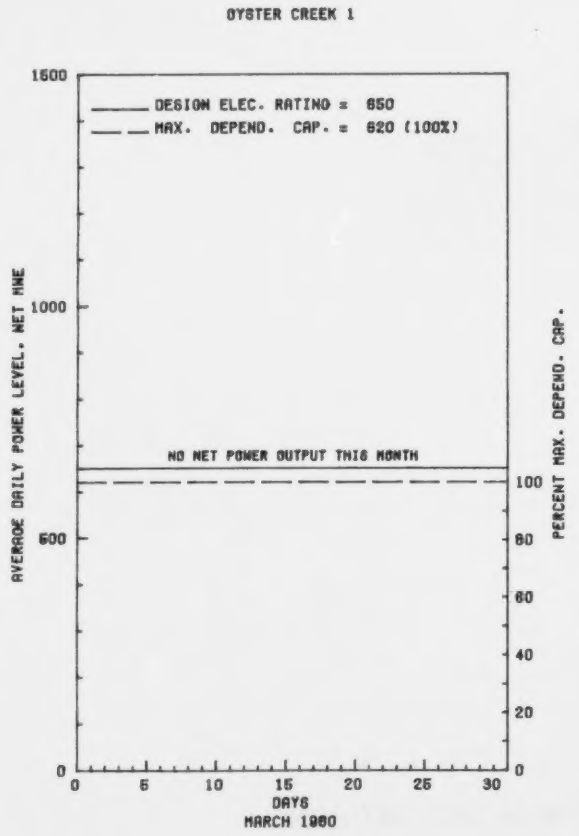
- 50-219/80-07 - Feb 12-15, 19-20 and 26-29: Routine, onsite, unannounced inspection by two region-based inspectors (98 hours) of 5 inspection areas: Licensee action on previous inspection findings; IE Bulletin and Circular followup; onsite licensee event followup; fire protection modification; and, independent inspection effort. Plant tours were also conducted. Noncompliances: None.

REVIEWED BY OMPA LOEB: John J. [illegible] **DATE:** 5-12-80

REVIEWED BY NRR DOR: Walter A. [illegible] **DATE:** 5-13-80

OYSTER CREEK

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-219
2. Utility Contact: C. M. McClain (201) 455-8748
3. Licensed Thermal Power (MWt): 1930
4. Nameplate Rating (Gross MWe): 687.5 x 0.8 = 550
5. Design Electrical Rating (Net MWe): 650
6. Maximum Dependable Capacity (Gross MWe): 650
7. Maximum Dependable Capacity (Net MWe): 620
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

ITEM 21 CALCULATED BY USING A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	2,184.0	90,048.0
12. Hours Reactor Was Critical	.0	101.5	68,740.2
13. Reactor Reserve Shutdown Hours	.0	.0	468.2
14. Hours Generator On-Line	.0	98.4	67,403.9
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	157,611	114,036,769
17. Gross Electrical Energy (MWH)	0	53,670	38,935,985
18. Net Electrical Energy (MWH)	0	48,612	37,529,625
19. Unit Service Factor	.0	4.5	74.9
20. Unit Availability Factor	.0	4.5	74.9
21. Unit Capacity Factor (MDC Net)	.0	3.6	60.9
22. Unit Capacity Factor (DER Net)	.0	3.4	64.1
23. Unit Forced Outage Rate	.0	.0	6.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 05/22/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
1	01/05/80	S	744.0	C	4		ZZ	ZZZZZZ	REFUELING/MAINTENANCE OUTAGE.

SUMMARY

THE UNIT REMAINED SHUTDOWN THROUGHOUT THE REPORT PERIOD FOR REFUELING.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

PALISADES

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Michigan/Van Buren 5 miles S of South Haven
2.	Electric Reliability Council	East Central Area Reliability Coordination Agreement
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2530
5.	Design Electrical Rating (MWE Net)	805
6.	Date of Initial Criticality	May 24, 1971
7.	Date of Commercial Operation	December 31, 1971
8.	Condenser Cooling Method	Forced Draft Cooling Towers
9.	Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10.	Licensee	Consumers Power Company
11.	Corporate Address	212 W. Michigan Avenue Jackson, Michigan 49201
12.	Corporate Contact	R. B. DeWitt
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Combustion Engineering
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Senior Inspector	D.C. Boyd
19.	IE Resident Inspector	B. Jorgenson (616) 764-8971
20.	Licensing Project Manager	T. Wambach
21.	Docket Number	50-255
22.	License Number/Date of Issue	DPR-20, 3/24/71
23.	Public Document Room	Kalamazoo Public Library 315 South Rose Street Reference Department Kalamazoo, Michigan 49006

INSPECTION STATUS

ENFORCEMENT STATUS

TECHNICAL SPECIFICATION 3.6.1 STATES THAT CONTAINMENT INTEGRITY SHALL NOT BE VIOLATED UNLESS THE REACTOR IS IN THE COLD SHUTDOWN CONDITION. CONTRARY TO THE ABOVE, THE LICENSEE ON NUMEROUS AND LENGTHY OCCASIONS FROM APRIL, 1978, TO SEPTEMBER, 1979, OPERATED THE REACTOR IN OTHER MODES, INCLUDING FULL-POWER OPERATIONS, WITH CONTAINMENT INTEGRITY VIOLATED BY THE LOCKED-OPEN CONDITION OF BOTH SERIES MANUAL ISOLATION VALVES IN THE CONTAINMENT PURGE EXHAUST BYPASS LINE.
(7915 VIOLATION)

TECHNICAL SPECIFICATION 3.6.3 REQUIRES THAT PRIOR TO GOING CRITICAL AFTER A REFUELING OUTAGE, AN ADMINISTRATIVE CHECK WILL BE MADE TO CONFIRM THAT ALL MANUAL CONTAINMENT ISOLATION VALVES ARE CLOSED AND LOCKED. CONTRARY TO THE ABOVE, DUE TO A DEFICIENT CONTAINMENT INTEGRITY VERIFICATION PROCEDURE, THE PLANT HAS BEEN TAKEN CRITICAL AFTER EACH OF ITS REFUELING OUTAGES WITHOUT CONFIRMATION THAT TWO MANUAL CONTAINMENT ISOLATION VALVES FOR THE CONTAINMENT PURGE EXHAUST BYPASS WERE CLOSED AND LOCKED AS REQUIRED.
(7915 INFRACTION)

TECHNICAL SPECIFICATION 6.8.1.C REQUIRES IMPLEMENTATION OF PROCEDURES FOR SURVEILLANCE AND TESTING OF SAFETY-RELATED EQUIPMENT. HEALTH PHYSICS PROCEDURE H.P.6.27 GOVERNS TESTING OF IN-PLACE HEPA AND CHARCOAL FILTERS IN SAFETY-RELATED SYSTEMS PURSUANT TO TECHNICAL SPECIFICATION TABLE 4.2.2, AND REQUIRES NOTIFICATION AND SIGN OFF BY THE SHIFT SUPERVISOR AND THE PLANT HEALTH PHYSICIST ON COMPLETION OF TESTING. CONTRARY TO THE ABOVE, THE REQUIRED NOTIFICATION AND SIGN OFFS WERE NOT COMPLETED AFTER THE LICENSEE TESTED FILTERS IN THE CONTAINMENT VENTILATION EXHAUST BYPASS LINE UNDER H.P.6.27 ON APRIL 4-7, 1978.
(7915 INFRACTION)

FAILURE OF LICENSEE TO COMPLY WITH 10 CFR 70.54 AND REFERENCED INSTRUCTION FOR COMPLETION AND DISTRIBUTION OF FORMS NRC-741.
(8001 DEFICIENCY)

10 CFR 50, APPENDIX J, SECTION IV.A REQUIRES, IN PART "...REPLACEMENT OF A COMPONENT WHICH IS PART OF THE PRIMARY REACTOR CONTAINMENT BOUNDARY,...SHALL BE FOLLOWED BY A TYPE A, TYPE B, OR TYPE C TEST, AS APPLICABLE..." CONTRARY TO THE ABOVE, THE LICNSEE PERIODICALLY REMOVES A PIPE CAP ON THE CONTAINMENT PRESSURE INSTRUMENT LINES TO TEST AND CALIBRATE THE PRESSURE TRANSMITTERS AND SWITCHES. ALTHOUGH THE PIPE CAP IS PART OF THE PRIMARY CONTAINMENT BOUNDARY, NO LEAK TEST WAS PERFORMED AFTER REPLACEMENT OF THE CAP FOLLOWING INSTRUMENT TESTING ON JUNE 5 AND ON JULY 2, 1979. THE REACTOR WAS OPERATED AT POWER AFTER BOTH TESTS.
(8002 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

- Reactor shutdown on September 10, 1979, to begin a 18-week scheduled refueling and maintenance outage. Start-up tentatively scheduled for late April 1980.

•LAST IE SITE INSPECTION DATE: December 1979 and January 1980

- INSPECTION REPORT NO: 80-02

SUMMARY

INSPECTION SUMMARY

- Inspection on January 23-25, (80-01): Licensee Action on Previous Inspection Findings, Facility Organization and Operation, Measurements and Control, Shipping and Receiving, Storage and Internal Control, Inventory, Records and Reports and Management of Nuclear Material Control Systems. The inspection involved 42 inspector hours onsite by two NRC inspectors and was begun during the regular hours. The licensee was found to be in compliance with NRC requirements in the eight areas examined during the inspection except for the following one item: Failure of licensee to comply with 10 CFR 70.54 and referenced instruction for completion and distribution of Forms NRC-741.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-32/ 03L-0	07/25/79	08/14/79	30 Day LER	High secondary system conductivity and sodium.

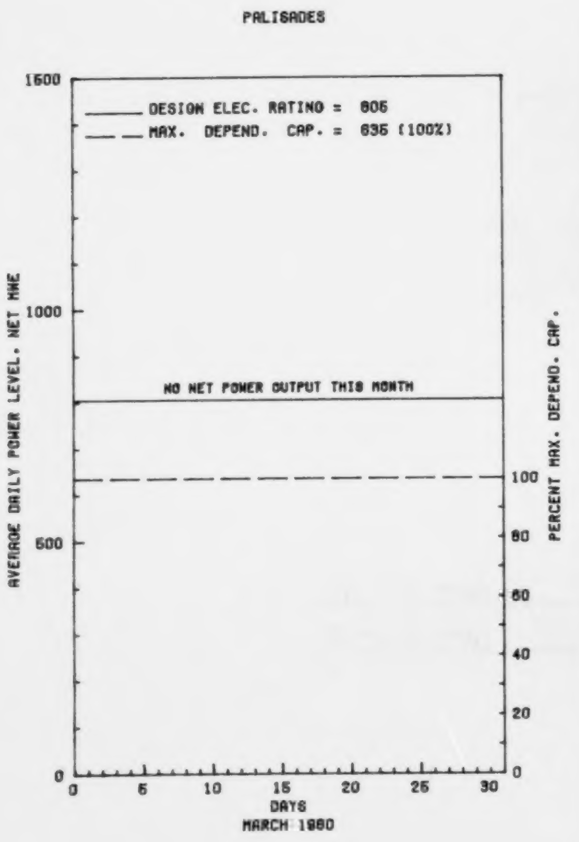
- Inspection during December 1979 and January 1980, (80-02): Routine, resident inspection program activities including operations, reportable occurrences, action on IE Bulletins and followup to previously identified items. Special inspection of licensee activities pursuant to NRC Order Modifying License DPR-20 dated November 9, 1979. The inspection involved 379 inspector hours onsite by three NRC inspectors. A meeting was held with local officials on January 14, 1980, to brief them on the NRC Resident Inspector Program and to introduce to them the Resident Inspector assigned at the Palisades Plant. Of the four areas examined, no items of noncompliance were identified in three areas. One item of noncompliance (failure to leak-test on restoration of primary containment boundary) was identified in the remaining area of inspection.

REVIEWED BY OMPA LOEB: AmR Beebe **DATE:** 05/12/80

REVIEWED BY NRR DOR: T. V. Wambach **DATE:** 5/13/80

PALISADES

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-255
2. Utility Contact: P. J. Bodtke (616) 764-8913
3. Licensed Thermal Power (MWt): 2530
4. Nameplate Rating (Gross MWe): 955 x 0.85 = 812
5. Design Electrical Rating (Net MWe): 805
6. Maximum Dependable Capacity (Gross MWe): 675
7. Maximum Dependable Capacity (Net MWe): 635
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	2,184.0	72,615.0
12. Hours Reactor Was Critical	.0	.0	39,564.8
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	.0	.0	37,426.5
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	0	71,511,480
17. Gross Electrical Energy (MWH)	0	0	22,032,230
18. Net Electrical Energy (MWH)	0	0	20,670,775
19. Unit Service Factor	.0	.0	51.5
20. Unit Availability Factor	.0	.0	51.5
21. Unit Capacity Factor (MDC Net)	.0	.0	44.8
22. Unit Capacity Factor (DER Net)	.0	.0	35.4
23. Unit Forced Outage Rate	.0	.0	36.2

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 04/28/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
13	09/08/79	S	744.0	D	4				TMI MODIFICATIONS AND ANCHOR BOLT INSPECTIONS/REPAIRS ACCORDING TO IE BULLETIN 79-02.

SUMMARY

THE UNIT REMAINED SHUTDOWN THROUGHOUT THE REPORT PERIOD FOR SEISMIC PIPE HANGER/ANCHOR BOLT INSPECTIONS AND REPAIRS, AND FOR TMI-RELATED MODIFICATIONS.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

PEACH BOTTOM 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Pennsylvania/York 19 miles S of Lancaster
2.	Electric Reliability Council	Mid-Atlantic Area Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	3293
5.	Design Electrical Rating (MWE Net)	1065
6.	Date of Initial Criticality	September 16, 1973
7.	Date of Commercial Operation	July 5, 1974
8.	Condenser Cooling Method	Once Through with Cooling Towers
9.	Condenser Cooling Water Source	Conowingo Pond/Susquehanna River

Utility & Contractor Information

10.	Licensee	Philadelphia Electric Company
11.	Corporate Address	2301 Market Street Philadelphia, Pennsylvania 19101
12.	Corporate Contact	S. L. Daltroff, Vice President Electric Production
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	GE
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	IE Senior Inspector	E. McCabe
19.	IE Resident Inspector	C. Cowgill 717-456-7614
20.	Licensing Project Manager	D. Verrelli
21.	Docket Number	50-277
22.	License Number/Date of Issue	DPR-44, 8/8/73
23.	Public Document Room	Government Publications Section State Library of Pennsylvania Education Building Commonwealth and Walnut Street Harrisburg, Pennsylvania 17126

INSPECTION STATUS

ENFORCEMENT STATUS

ON 1/10/80, UNSEALED PENETRATIONS THROUGH FIRE BARRIER WALLS WERE OBSERVED IN THE DIESEL GENERATOR ROOM AND IN THE CONTROL ROOM WITHOUT THE BENEFIT OF FIRE WATCH IN THE AFFECTED AREAS.
(8002 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

MANAGERIAL ITEMS:

+ A group of National Security Agency members toured the facility on March 19, 1980.

+ Curtis J. Cowgill, III, Reactor Resident Inspector, has been designated to relieve E. G. Greenman as Senior Resident Inspector.

PLANT STATUS:

+ The reactor operated at a nominal 100 percent power during the month. A 4-day outage was commenced on 3/7/80 for a recirculation pump seal replacement. The unit was shutdown on 3/21/80 for a planned 104 day refueling outage. Representative offgas and combined Unit 2 and Unit 3 stack gas release rates were 12,100 microcuries per second and 1010 microcuries per second, respectively.

+ LAST IE SITE INSPECTION DATE: 3/31/80

+ INSPECTION REPORT NO: 50-277/80-05

SUMMARY

INSPECTION SUMMARY

+ 50-277/79-21 - Sep 17-21: Routine unannounced inspection by a region-based inspector of radiation protection including licensee action on IE Circulars, supplies and equipment, training, exposure control, respiratory protection, posting and control, surveys, and radioactive material control. Upon arrival a tour of the radiation control area was conducted to observe radiation safety practices. Noncompliance: One (Failure to evaluate exposures to airborne activity).

+ 50-277/80-04 - Feb 23, 25-28: Special onsite regular and backshift inspection by the resident inspectors (26 hours). Areas inspected included circumstances of the valving out of a seismically qualified air supply to five Unit 3 containment ventilation valves, a blocking error which resulted in valving out the air supply to the valve disc seals for two valves, valve modifications, and followup on prior associated LERs. Noncompliances: Three (Failure to establish adequate approved procedures to assure seismically qualified air supply systems for five containment isolation valves; Failure to take adequate and timely corrective action to assure that containment ventilation valves would not be opened beyond the point whereby closure against post-LOCA conditions could be assured; and, Failure to follow procedures - four examples related to modifications and the blocking and permit system).

+ 15000039/80-01 - Jan 31-Feb 1: Chem-Nuclear Waste disposal site, Barnwell, South Carolina for unannounced inspections of licensee-shippers. Areas inspected included each licensee-shipper vehicle inspection for compliance with Department of Transportation (DOT) and NRC regulations as follows: (1) shipping paper requirements; (2) DOT Placarding requirements; (3) radiation levels; (4) removable contamination; (5) DOT marking and labeling requirements for packages; (6) DOT and NRC requirements for external package features; and, (7) prohibited articles or contents. Noncompliances: None.

+ 15000039/80-2A - Feb 6-8: Chem-Nuclear Waste disposal site, Barnwell, South Carolina for unannounced inspections of licensee-shippers. Areas inspected included each licensee-shipper vehicle for compliance with Department of Transportation (DOT) and NRC regulations as follows: (1) shipping paper requirements; (2) DOT Placarding requirements; (3) radiation levels; (4) removable contamination; (5) DOT marking and labeling requirements for packages; (6) DOT and NRC requirements for external package features; and, (7) prohibited articles or contents. Noncompliances: None.

REPORTS RECEIVED FROM LICENSEE

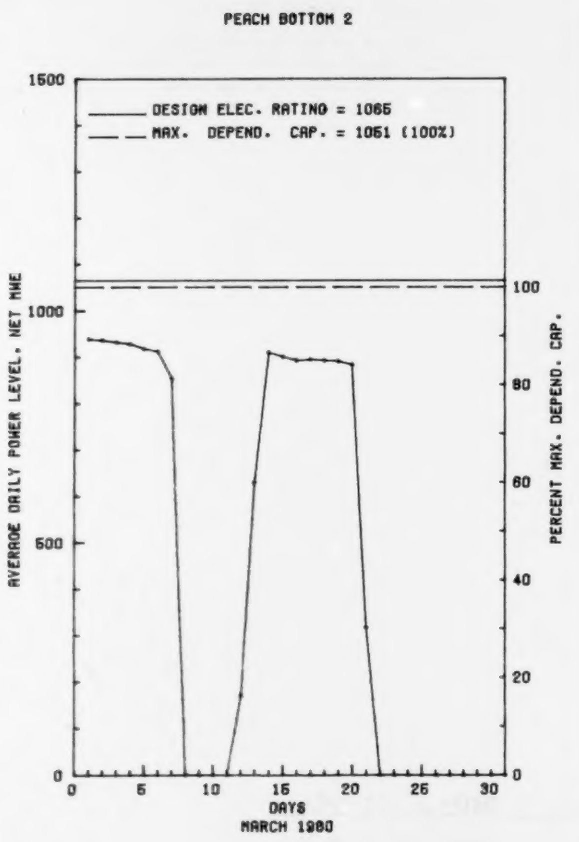
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
80-005/ 03L	01/28/80	02/25/80	30 Day LER	Diesel driven fire pump outlet below TS value
80-007/ 03L	03/06/80	03/26/80	30 Day LER	During surveillance testing one pressure switch for the 'C' channel low pressure signal to initiate closure of MSIV's was found outside required value by less than one percent and was reset

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 5-12-80

REVIEWED BY NRR DOR: **DATE:** 5-13-80

PEACH BOTTOM 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-277
2. Utility Contact: W. M. Alden (215) 841-5022
3. Licensed Thermal Power (MWt): 3293
4. Nameplate Rating (Gross MWe): 1280 x 0.9 = 1152
5. Design Electrical Rating (Net MWe): 1065
6. Maximum Dependable Capacity (Gross MWe): 1098
7. Maximum Dependable Capacity (Net MWe): 1051
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	2,184.0	50,328.0
12. Hours Reactor Was Critical	398.0	1,441.0	39,196.0
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	386.0	1,390.2	38,373.0
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,049,354	3,831,770	110,597,414
17. Gross Electrical Energy (MWH)	344,860	1,255,980	36,314,210
18. Net Electrical Energy (MWH)	327,650	1,203,390	34,819,175
19. Unit Service Factor	51.9	63.7	76.2
20. Unit Availability Factor	51.9	63.7	76.2
21. Unit Capacity Factor (MDC Net)	41.9	52.4	65.8
22. Unit Capacity Factor (DER Net)	41.4	51.7	65.0
23. Unit Forced Outage Rate	.0	.9	6.1

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 06/21/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
8	03/08/80	S	104.1	B	2		CB	PUMPXX	2A REACTOR RECIRC. PUMP SEAL REPAIR.
9	03/21/80	S	253.9	C	2		RC	FUELXX	REFUEL OUTAGE.

SUMMARY

THE UNIT OPERATED IN AN END-OF-CYCLE COASTDOWN UNTIL 03/21/80, WHEN IT WAS SHUTDOWN FOR REFUELING. THE UNIT WAS ALSO SHUTDOWN FROM 03/07/80 TO 03/12/80 FOR 2A RECIRCULATION PUMP SEAL REPAIR AND DUE TO PROBLEMS WITH THE MINIMUM FLOW VALVE ON THE RESIDUAL HEAT REMOVAL SYSTEM.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

PEACH BOTTOM 3

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Pennsylvania/York 19 miles S of Lancaster
2.	Electric Reliability Council	Mid-Atlantic Area Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	3293
5.	Design Electrical Rating (MWE Net)	1065
6.	Date of Initial Criticality	August 7, 1974
7.	Date of Commercial Operation	December 23, 1974
8.	Condenser Cooling Method	Once Through with Cooling Towers
9.	Condenser Cooling Water Source	Conowingo Pond/Susquehanna River

Utility & Contractor Information

10.	Licensee	Philadelphia Electric Company
11.	Corporate Address	2301 Market Street Philadelphia, Pennsylvania 19101
12.	Corporate Contact	S. L. Daltroff, Vice President Electric Production
13.	Architect/Engineer	Bechtal
14.	Nuclear Steam Supply System	GE
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	IE Senior Inspector	E. McCabe
19.	IE Resident Inspector	C. Cowgill (717-456-7614)
20.	Licensing Project Manager	D. Verrelli
21.	Docket Number	50-278
22.	License Number/Date of Issue	DPR-56, 7/2/74
23.	Public Document Room	Government Publications Section State Library of Pennsylvania Education Building Commonwealth and Walnut Street Harrisburg, Pennsylvania 17126

INSPECTION STATUS

ENFORCEMENT STATUS

INTAKES IN EXCESS OF THOSE WHICH WOULD RESULT FROM INHALATION FOR 2 HOURS IN A DAY AT UNIFORM CONCENTRATIONS SPECIFIED IN APPENDIX B, TABLE I, COLUMN 1 OF 10 CFR 20 WERE NOT MADE FOR 3 INDIVIDUALS EXPOSED ON 9/18/79.
(7923 INFRACTION)

ON 1/10/80, UNSEALED PENETRATIONS THROUGH FIRE BARRIER WALLS WERE OBSERVED IN THE DIESEL GENERATOR ROOM AND IN THE CONTROL ROOM WITHOUT THE BENEFIT OF FIRE WATCH IN THE AFFECTED AREAS.
(8002 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

+ A startup transformer tripped on 3/5/80 due to spurious initiation of a fire protection deluge system. Failure of one of the loads to shift to a backup power supply resulted in a containment isolation signal. During resetting of the signal, main steam isolation valve began to drift shut due to isolation of the instrument nitrogen supply and a reactor trip resulted.

MANAGERIAL ITEMS:

+ A group of National Security Agency members toured the facility on March 19, 1980.

+ Curtis J. Cowgill, III, Resident Reactor Inspector, has been designated to relieve E. G. Greenman as Senior Resident Inspecter.

PLANT STATUS:

+ The reactor operated at a nominal 100 percent power during the month. One reactor shutdown occurred on 3/5/80 following loss of a transformer. The planned outage continued until 3/116/80 as repairs to isolation valves in the RHR and RCIC system were accomplished. Representative offgas and combined Unit 2 and Unit 3 stack gas release rates were 3920 microcuries per second and 1010 microcuries per second, respectively.

+ LAST IE SITE INSPECTION DATE: 3/31/80

+ INSPECTION REPORT NO: 50-278/80-05

SUMMARY

INSPECTION SUMMARY

+ 50-278/79-23 - Sep 17-21: Routine, unannounced inspection by a region-based inspector of radiation protection during refueling including licensee action on IE Circulars, supplies and equipment, training, exposure control, respiratory protection, posting and control, surveys, and radioactive material control. Upon arrival a tour of the radiation control area was conducted to observe radiation safety practices (20 Hours). Noncompliances: One (Failure to evaluate exposures to airborne activity).

+ 50-278/80-04 - Feb 23, 25-28: Special onsite regular and backshift inspection by the resident inspectors: (37.5 hours). Areas inspected included circumstances of the valving out of a seismically qualified air supply to five Unit 3 containment ventilation valves, a blocking error which resulted in valving out the air supply to the valve disc seals for two valves, valve modifications and followup on prior associated LERs. Noncompliances: Three (Failure to establish adequate approved procedures to assure seismically qualified air supply system for five containment isolation valves; Failure to take adequate and timely corrective actions to assure that containment ventilation valves would not be opened beyond the point whereby closure against post-LOCA conditions could be assured; and Failure to follow procedures -- four examples related to modification and the blocking and permit system).

+ 15000039/80-01 - Jan 31-Feb 1: Chem-Nuclear Waste disposal site, Barnwell, South Carolina for unannounced inspections of licensee-shippers. Areas inspected included each licensee-shipper vehicle for compliance with Department of Transportation (DOT) and NRC regulations as follows: (1) shipping paper requirements; (2) DOT Placarding requirements; (3) radiation levels; (4) removable contamination; (5) DOT marking and labeling requirements for packages; (6) DOT and NRC requirements for external package features; and, (7) prohibited articles or contents. Noncompliances: None.

+ 15000039/80-2A - Feb 6-8: Chem-Nuclear Waste disposal site, Barnwell, South Carolina for unannounced inspections of licensee-shippers. Areas inspected included each licensee-shipper vehicle for compliance with Department of Transportation (DOT) and NRC regulations as follows: (1) shipping paper requirements; (2) DOT Placarding requirements; (3) radiation levels; (4) removable contamination; (5) DOT marking and labeling requirements for packages; (6) DOT and NRC requirements for external package features; and, (7) prohibited articles or contents. Noncompliances: None.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

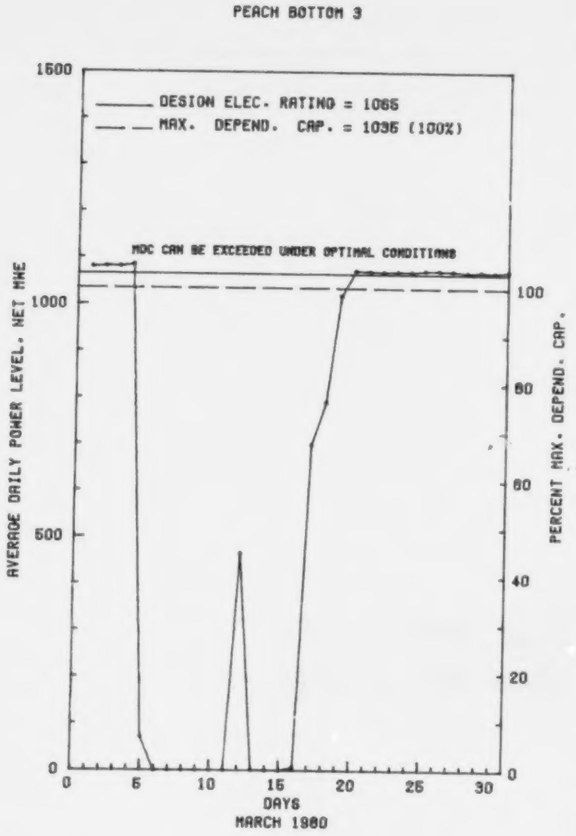
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
80-004/ 03L	02/05/80	03/06/80	30 Day LER	Reactor water conductivity of 12 6 nmho/cm (limit is 10), with shutdown effected until cleanup accomplished
80-005/ 03L	02/07/80	03/04/80	30 Day LER	During surveillance the setpoint of a differential pressure switch for recirc steam was found set 10 inches above allowable due to setpoint drift.
80-006/ 03L	02/13/80	03/12/80	30 Day LER	RCIC automatic flow control was inoperable for 100 hours due to a faulty flow controller circuit board
80-007/ 01T	02/21/80	03/06/80	24 Hours	Seismically qualified air supply to certain containment isolation valves was valved out of service. Additional blocking error was identified. Valving and blocking errors were cor upon discoveru
80-008/ 01T	03/05/80	03/19/80	24 Hours	Emergency power source failure without E-334 480V load center recovery

REVIEWED BY OMPA LOEB: DATE: 5-12-80

REVIEWED BY NRR DOR: DATE: 5-13-80

PEACH BOTTOM 3

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-278
2. Utility Contact: W. M. Alden (215) 841-5022
3. Licensed Thermal Power (MWt): 3293
4. Nameplate Rating (Gross MWe): 1280 x 0.9 = 1152
5. Design Electrical Rating (Net MWe): 1065
6. Maximum Dependable Capacity (Gross MWe): 1098
7. Maximum Dependable Capacity (Net MWe): 1035
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	2,184.0	46,224.0
12. Hours Reactor Was Critical	514.0	1,667.0	36,513.0
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	482.7	1,584.5	35,516.3
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,490,549	4,993,467	99,212,621
17. Gross Electrical Energy (MWH)	502,320	1,681,050	32,266,910
18. Net Electrical Energy (MWH)	483,602	1,623,513	30,968,313
19. Unit Service Factor	64.9	72.6	76.8
20. Unit Availability Factor	64.9	72.6	76.8
21. Unit Capacity Factor (MDC Net)	62.8	71.8	64.7
22. Unit Capacity Factor (DER Net)	61.0	69.8	62.9
23. Unit Forced Outage Rate	26.0	9.7	7.0

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
5	03/05/80	F	169.6	A	3		EA	CKTBRK	LOSS OF START UP FEED WITH MSIV DRIFT FROM FULL OPEN.
6	03/13/80	S	91.7	B	2		HA	VALVEX	LEAKAGE OF FEEDWATER FROM RCIC INJECTION CHECK VALVE.
7	03/17/80	S	.8	H	4		RC	ZZZZZZ	LOAD DROP FOR CONTROL ROD ADJUSTMENT.

SUMMARY

THE UNIT WAS SHUTDOWN BETWEEN 03/05/80 AND 03/11/80 FOLLOWING A REACTOR SCRAM CAUSED BY THE MAIN STEAMLINE ISOLATION VALVES DRIFTING CLOSED AFTER LOSS OF OFFSITE POWER. THE UNIT WAS ALSO SHUTDOWN FROM 03/13/80 TO REPAIR A LEAK IN THE REACTOR CORE ISOLATION TESTABLE CHECK VALVE. A BRIEF LOAD REDUCTION OCCURRED ON 03/17/80 FOR CONTROL ROD ADJUSTMENT.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

PILGRIM

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Massachusetts/Plymouth 4 miles SE of Plymouth
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	BWR
4.	Licenses Power Level (MWT)	1998
5.	Design Electrical Rating (MWE Net)	655
6.	Date of Initial Criticality	June 16, 1972
7.	Date of Commercial Operation	December 1972
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Cape Cod Bay

Utility & Contractor Information

10.	Licensee	Boston Edison Company
11.	Corporate Address	800 Boylston Street Boston, Massachusetts 02199
12.	Corporate Contact	A. Corey, Vice President
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	GE
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	IE Senior Inspector	E. McCabe
19.	IE Principal Inspector	K. Roberts
20.	Licensing Project Manager	J. Hannon
21.	Docket Number	50-293
22.	License Number/Date of Issue	DPR-35, 6/8/72
23.	Public Document Room	Plymouth Public Library North Street Plymouth, Massachusetts 02360

INSPECTION STATUS

ENFORCEMENT STATUS

A SHIPMENT OF LICENSED RADIOACTIVE MATERIAL WAS MADE BY THE LICENSEE TO BARNWELL, SOUTH CAROLINA ON 12/14/79, IN AN EXCLUSIVE USE VEHICLE, IN WHICH THE DOSE RATE AS MEASURED FROM THE LICENSED MATERIAL WAS 15-17 MILLIREM PER HOUR AT 2 METERS (6 FEET) FROM THE VERTICAL PLANES PROJECTED BY THE OUTER LATERAL SURFACES OF THE VEHICLE. UPON RECEIPT OF THE MATERIAL AT BARNWELL, SOUTH CAROLINA ON 12/17/79. (7923 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

- Indication of cracks on three jet pump holddown bars. Bars to be replaced during existing refueling outage.

PLANT STATUS:

- Shutdown for refueling until early May.

- LAST IE SITE INSPECTION DATE: 4/1/80
- INSPECTION REPORT NO: 50-293/80-13

SUMMARY

INSPECTION SUMMARY

- 50-293/80-02 - Jan 17-18: Special announced inspection by one region-based inspector of the licensees shipment of radioactive waste to Barnwell, South Carolina. The inspection involved 15 hours onsite. No items of noncompliance were identified.
- 50-293/80-03 - Jan 22-24: Routine, unannounced inspection of the torus modification and the inservice inspection program. The inspection involved 20 hours onsite by two region-based inspectors. No items of noncompliance were identified.
- 50-293/80-06 - Feb 2: Management meeting held at NRC request to discuss details of establishing the NRC Resident Office at Pilgrim Nuclear Power Station.
- 50-293/80-07 - Feb 20-22: Routine, unannounced inspection of the torus modification and inservice inspection. The inspection involved 36 hours onsite by two region-based inspectors. No items of noncompliance were identified.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

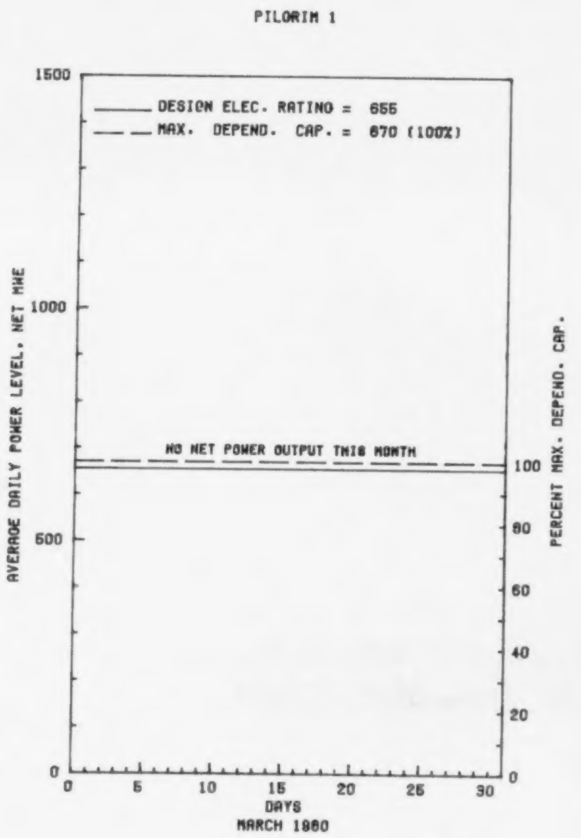
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-052/03X	07/05/79	02/15/80	30 Day LER	HPCI turbine compartment fire detector inoperable
79-053/04X	07/10/79	02/15/80	Special	Exceeded allowable condenser delta temperature
79-054/03X	08/07/79	02/15/80	30 Day LER	Temporary loss of shutdown transformer
80-008/01T	03/08/80	03/20/80	14 Day LER	Movement of spent fuel with diesel generators inoperable and secondary containment not intact
80-009/04T	03/10/80	03/18/80	14 Day LER	Concentrations of Cr-51 and Co-60 in mussels are in excess of 10 times lower limit of detection

REVIEWED BY OMPA LOEB: John J. Rizzo DATE: 5-12-80

REVIEWED BY NRR DOR: John N Hannon DATE: 5-14-80

PILGRIM

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-293
2. Utility Contact: G. G. Whitney (617) 746-7900
3. Licensed Thermal Power (MWt): 1998
4. Nameplate Rating (Gross MWe): 780 x 0.87 = 678
5. Design Electrical Rating (Net MWe): 655
6. Maximum Dependable Capacity (Gross MWe): 690
7. Maximum Dependable Capacity (Net MWe): 670
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	2,184.0	64,080.0
12. Hours Reactor Was Critical	.0	96.1	45,077.0
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	.0	96.1	43,648.6
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	181,152	73,272,264
17. Gross Electrical Energy (MWH)	0	61,340	24,226,174
18. Net Electrical Energy (MWH)	0	58,860	23,264,983
19. Unit Service Factor	.0	4.4	68.1
20. Unit Availability Factor	.0	4.4	68.1
21. Unit Capacity Factor (MDC Net)	.0	4.0	54.2
22. Unit Capacity Factor (DER Net)	.0	4.1	55.4
23. Unit Forced Outage Rate	.0	.0	10.6

24. Shutdowns Scheduled Over Next 6 Months (Type, Date and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 05/15/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
1	01/05/80	S	744.0	C	4		RC	FUELXX	REFUELING CONTINUES.

SUMMARY

THE UNIT REMAINED SHUTDOWN THROUGHOUT THE REPORT PERIOD FOR REFUELING.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

POINT BEACH 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Wisconsin/Manitowoc 15 miles N of Manitowoc
2.	Electric Reliability Council	Mid-America Interpool Network
3.	Type of Reactor	PWR
4.	Power Level (MWT)	1518
5.	Design Electrical Rating (MWE Net)	497
6.	Date of Initial Criticality	November 2, 1970
7.	Date of Commercial Operation	December 21, 1970
8.	Condenser Cooling Method	Straight Through
9.	Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10.	Licensee	Wisconsin Electric Power Co.
11.	Corporate Address	231 West Michigan Milwaukee, Wisconsin
12.	Corporate Contact	S. Burstein
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Senior Inspector	R. F. Warnick
19.	IE Resident Inspector	B.L. Jorgnesen
20.	Licensing Project Manager	C. M. Trammell
21.	Docket Number	50-266
22.	License Number/Date of Issue	DPR-24 issued 10/06/70
23.	Public Document Room	Document Department, Library University of Wisconsin-Stevens Point Stevens Point, Wisconsin 54481 Manitowoc Public Library Manitowoc, Wisconsin (Temporary LPDR for spent fuel pool capacity increase amendment only

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

+ The unit was offline in March for inservice inspection of steam generator tubes.

+LAST IE SITE INSPECTION DATE: January 22-24, 1980

+ INSPECTION REPORT NO: 80-02

SUMMARY

INSPECTION SUMMARY

+ Inspection on January 22-24, (80-02): Routine, unannounced inspection January 22-24, 1980 to review plant operations, organization and administration, quality assurance, review and audits, and records. The inspection involved 48 inspector hours onsite by two NRC inspectors. Of the five areas inspected, no items of noncompliance were identified.

REPORTS RECEIVED FROM LICENSEE

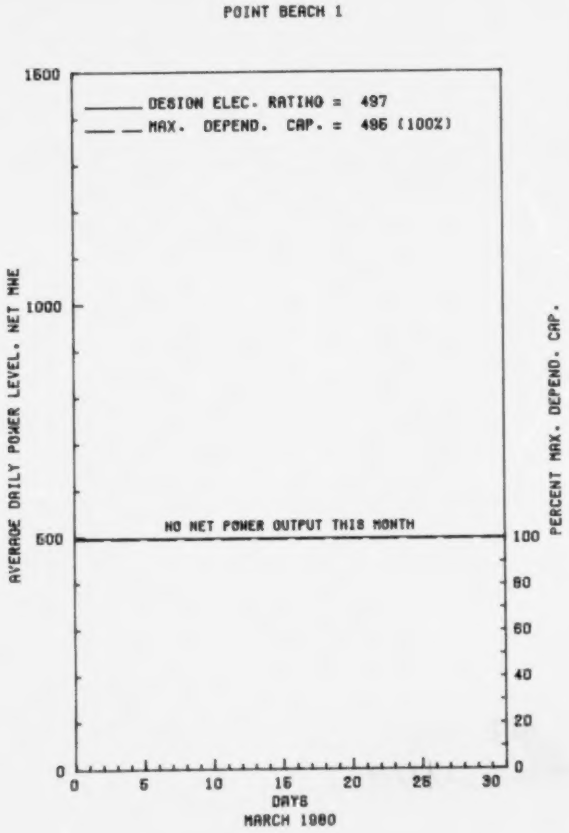
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
80-01/ 01T-0	02/02/80	02/15/80	14 Day LER	Pressure transmitters isolation block closed.
80-02/ 01T-0	03/11/80	03/25/80	14 Day LER	Secondary-to-primary hydrostatic leak test of the SG performed.

REVIEWED BY OMPA LOEB: DATE: 5/12/80

REVIEWED BY NRR DOR: CM Trammell DATE: 5/12/80

POINT BEACH 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-266
2. Utility Contact: C.W. FAY (414) 277-2811
3. Licensed Thermal Power (MWt): 1518
4. Nameplate Rating (Gross MWe): 582 x 0.9 = 524
5. Design Electrical Rating (Net MWe): 497
6. Maximum Dependable Capacity (Gross MWe): 519
7. Maximum Dependable Capacity (Net MWe): 495
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): 390
10. Reasons for Restrictions, If Any: INHIBIT CORROSION OF STEAM GENERATOR TUBES

ITEM 21 CALCULATED BY USING A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	2,184.0	82,416.0
12. Hours Reactor Was Critical	115.0	1,544.7	68,061.2
13. Reactor Reserve Shutdown Hours	7.3	17.6	599.8
14. Hours Generator On-Line	.0	1,415.7	65,809.7
15. Unit Reserve Shutdown Hours	122.3	146.6	749.0
16. Gross Thermal Energy (MWH)	0	1,696,614	92,449,409
17. Gross Electrical Energy (MWH)	0	554,070	31,187,820
18. Net Electrical Energy (MWH)	-2,628	522,615	29,712,793
19. Unit Service Factor	.0	64.8	79.9
20. Unit Availability Factor	16.4	71.5	80.8
*21. Unit Capacity Factor (MDC Net)	.0	48.3	74.1
22. Unit Capacity Factor (DER Net)	.0	48.1	72.5
23. Unit Forced Outage Rate	.0	.0	3.6

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): 07/03/80 - STEAM GEN LEAK TESTING & EDDY CURRENT INSPECTION.
25. If Shut Down At End of Report Period, Estimated Date of Startup: 04/06/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
1	02/28/80	S	744.0	D	4	80-002	CB	HTEXCH	AS REQUIRED BY THE CONFIRMATORY ORDER OF 11/30/79, THE UNIT WAS TAKEN OFF LINE ON 2/28/80 FOR THE 60 EFFECTIVE FULL POWER DAY STEAM GENERATOR TESTING. WORK DID NOT START ON UNIT 1 UNTIL 3/10/80 BECAUSE OF A FORCED UNIT 2 OUTAGE. AN 800 PSI SECONDARY-TO-PRIMARY LEAK TEST OF BOTH GENERATORS WAS PERFROMED ON 3/11/80 AND AN EDDY CURRENT INSPECTION WAS CONDUCTED.

SUMMARY

THE UNIT REMAINED SHUTDOWN THROUGHOUT THE REPORT PERIOD FOR STEAM GENERATOR LEAK TESTING AND EDDY CURRENT INSPECTION.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

POINT BEACH 2

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Wisconsin/Manitowoc 15 miles N of Manitowoc
2.	Electric Reliability Council	Mid-America Interpool Network
3.	Type of Reactor	PWR
4.	Power Level (MWT)	1518
5.	Design Electrical Rating (MWE Net)	497
6.	Date of Initial Criticality	May 30, 1972
7.	Date of Commercial Operation	October 1, 1972
8.	Condenser Cooling Method	Straight Through
9.	Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10.	Licensee	Wisconsin Electric Power Co.
11.	Corporate Address	231 West Michigan Milwaukee, Wisconsin
12.	Corporate Contact	S. Burstein
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Senior Inspector	R. F. Warnick
19.	IE Resident Inspector	B.L. Jorgnesen
20.	Licensing Project Manager	C. M. Trammell
21.	Docket Number	50-301
22.	License Number/Date of Issue	DPR-27 issued 11/16/71
23.	Public Document Room	Document Department, Library University of Wisconsin-Stevens Point Stevens Point, Wisconsin 54481 Manitowoc Public Library Manitowoc, Wisconsin (Temporary LPDR for spent fuel pool capacity increase amendment only

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

+ The unit was returned to service on March 13, 1980 following repair of a steam generator tube leak. The unit is in stretchout with anticipated shutdown date for refueling of April 11, 1980.

+LAST IE SITE INSPECTION DATE: January 22-24, 1980

+ INSPECTION REPORT NO: 80-01

SUMMARY

INSPECTION SUMMARY

+ Inspection on January 22-24, (80-01): Routine, unannounced inspection January 22-24, 1980 to review plant operations, organization and administration, quality assurance, review and audits, and records. The inspection involved 48 inspector hours onsite by two NRC inspectors. Of the five areas inspected, no items of noncompliance were identified.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
80-01/ 01T-0	02/21/80	02/28/80	14 Day LER	Momentary loss of the red instrument bus.

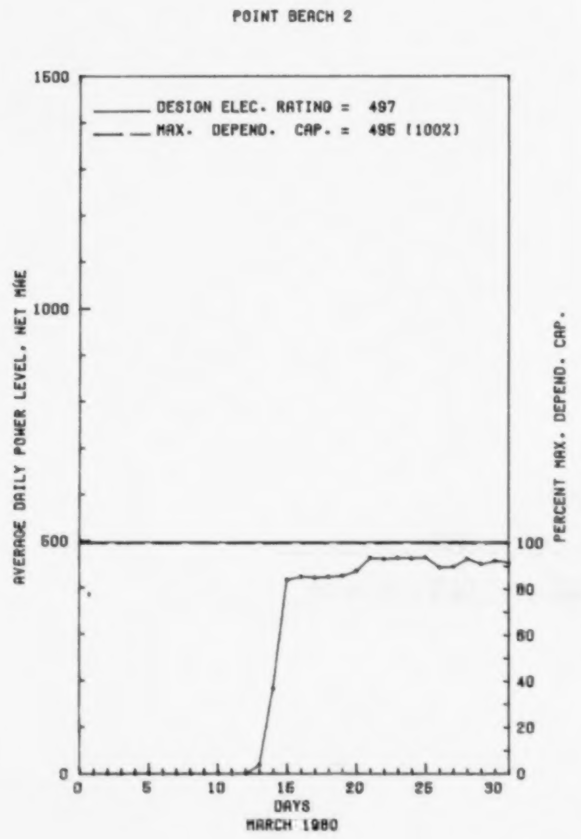
==

REVIEWED BY OMPA LOEB: John DATE: 5/12/80

REVIEWED BY NRR DOR: CM Trammell DATE: 5/12/80

POINT BEACH 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-301
2. Utility Contact: C.W. FAY (414) 277-2811
3. Licensed Thermal Power (MWt): 1518
4. Nameplate Rating (Gross MWe): 582 x 0.9 = 524
5. Design Electrical Rating (Net MWe): 497
6. Maximum Dependable Capacity (Gross MWe): 519
7. Maximum Dependable Capacity (Net MWe): 495

ITEM 21 CALCULATED BY USING A WEIGHTED AVERAGE

8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	2,184.0	67,201.0
12. Hours Reactor Was Critical	447.0	1,841.4	60,738.3
13. Reactor Reserve Shutdown Hours	.0	.0	156.1
14. Hours Generator On-Line	438.0	1,832.4	59,559.4
15. Unit Reserve Shutdown Hours	1.1	1.1	107.5
16. Gross Thermal Energy (MWH)	613,022	2,698,713	80,351,249
17. Gross Electrical Energy (MWH)	201,970	894,470	27,255,810
18. Net Electrical Energy (MWH)	191,808	853,180	25,928,307
19. Unit Service Factor	58.9	83.9	88.6
20. Unit Availability Factor	59.0	84.0	88.8
*21. Unit Capacity Factor (MDC Net)	52.1	78.9	78.7
22. Unit Capacity Factor (DER Net)	51.9	78.6	77.6
23. Unit Forced Outage Rate	41.1	16.1	2.0

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
REFUELING - 4/11/80 - 5 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
1	02/28/80	F	306.0	A	4	80-002	CB	HTEXCH	AN ORDERLY SHUTDOWN OF THE UNIT WAS COMPLETED AT 0224 HOURS ON 2/28/80 FOLLOWING CONFIRMATION OF PRIMARY-TO-SECONDARY LEAKAGE OF 1,428 GALLONS PER DAY IN THE "A" STEAM GENERATOR. A STATIC HEAD LEAK CHECK IDENTIFIED THE DEFECTIVE TUBE AND A SUBSEQUENT EDDY CURRENT INSPECTION PLACED THE DEFECT AT 10" ABOVE THE TUBE END. AN EDDY CURRENT INSPECTION OF BOTH STEAM GENERATORS WAS PERFORMED AND TUBES WITH DEFECTS ABOVE THE PLUGGING LIMIT WERE PLUGGED.

SUMMARY

OPERATION WAS ROUTINE FOLLOWING RETURN TO POWER ON 03/13/80 FROM AN EDDY CURRENT EXAMINATION OF THE STEAM GENERATOR, AND PLUGGING OF LEAKING AND DEFECTIVE TUBES.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

PRAIRIE ISLAND 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1. Location (state/county) (nearest population center)	Minnesota/Goodhue 28 miles SE of Minneapolis
2. Electric Reliability Council	Mid-Continent Area Reliability Coordination Agreement
3. Type of Reactor	PWR
4. Power Level (MWT)	1650
5. Design Electrical Rating (MWE Net)	530
6. Date of Initial Criticality	December 1, 1973
7. Date of Commercial Operation	December 16, 1973
8. Condenser Cooling Method	Cooling Towers
9. Condenser Cooling Water Source	Mississippi River

Utility & Contractor Information

10. Licensee	Northern States Power Company
11. Corporate Address	414 Nicollet Mall - 8th Floor Minneapolis, Minnesota 55401
12. Corporate Contact	G. H. Neils (Oper.) L. O. Meyer (Lic.)
13. Architect/Engineer	Pioneer Services & Eng. Co.
14. Nuclear Steam Supply System	Westinghouse
15. Constructor	Northern States Power Co.

Regulatory Information

16. IE Region responsible	III
17. IE Branch Chief	R. Heishman
18. IE Senior Inspector	R. F. Warnick
19. IE Resident Inspector	C. D. Feierabend 612-388-8209
20. Licensing Project Manager	R. Martin
21. Docket Number	50-282
22. License Number/Date of Issue	DPR-42 issued 8/9/73
23. Public Document Room	Environmental Conservation Library Minneapolis Public Library 300 Nicollet Mall Minneapolis, Minnesota 55401

INSPECTION STATUS

ENFORCEMENT STATUS

TECHNICAL SPECIFICATION 3.3.B.1 STATES THAT A REACTOR SHALL NOT BE MADE OR MAINTAINED CRITICAL NOR SHALL IT BE HEATED ABOVE 200 DEGREES F UNLESS THE FOLLOWING CONDITIONS ARE SATISFIED... C. THE SPRAY ADDITIVE TANK CONTAINS NOT LESS THAN 1950 GALLONS OF SOLUTION WITH A SODIUM HYDROXIDE CONCENTRATION OF NOT LESS THAN 30% BY WEIGHT. THIS IMPLIES THAT THE SPRAY ADDITIVE BE AVAILABLE FOR ADDITION TO THE CONTAINMENT SPRAY SYSTEM WHEN ACTUATED. CONTRARY TO THE ABOVE, FROM ABOUT 2330 ON JANUARY 3, 1980 TO 0600 JANUARY 6, 1980, MANUAL ISOLATION VALVES CA-1-2 AND CA-1-4 WERE CLOSED, ISOLATING THE SPRAY ADDITIVE TANK FROM THE CONTAINMENT SPRAY SYSTEM.
(8001 INFRACTION)

TECHNICAL SPECIFICATION 6.5 STATES THAT DETAILED WRITTEN PROCEDURES, INCLUDING THE APPLICABLE CHECKOFF LISTS AND INSTRUCTIONS, COVERING AREAS LISTED BELOW SHALL BE PREPARED AND FOLLOWED. THESE PROCEDURES AND CHANGES THERETO, EXCEPT AS SPECIFIED IN TS 6.5.D, SHALL BE REVIEWED BY THE OPERATIONS COMMITTEE AND APPROVED BY A MEMBER OF PLANT MANAGEMENT DESIGNATED BY THE PLANT MANAGER. A. PLANT OPERATIONS (1) INTEGRATED AND SYSTEM PROCEDURES FOR NORMAL STARTUP, OPERATION AND SHUTDOWN OF THE REACTOR AND ALL SYSTEMS AND COMPONENTS INVOLVING NUCLEAR SAFETY OF THE FACILITY. C1.3, UNIT SHUTDOWN PROCEDURE, CONTROLS ACTIONS FOR UNIT SHUTDOWN. (COMPONENT NUMBERS IDENTIFIED BY BRACKETS IDENTIFY UNIT 2 COMPONENTS). STEP 37 OF PROCEDURE C1.3 STATES IN PART: WHEN THE RC TEMPERATURE HAS BEEN REDUCED TO LESS THAN 200 DEGREES F (180 DEGREES F IF PRESSURIZER STEAM BUBBLE IS RETAINED): (A) PLACE NO. 11 AND 12 (21 AND 22) CONTAINMENT SPRAY PUMP CS IN PULLOUT-STOP; AND (B) REMOVE SAFEGUARD HOLD CARD. CLOSE AND ATTACH SECURE CARDS TO CA-1-2 (2CA-1-2), CV-31941, (CV-31939) OUTLET ISOL AND CA-1-4 (2CA-1-4), CV-31938 (CV-31940) OUTLET ISOL. DELIVER SAFEGUARDS HOLD CARD TO THE SHIFT SUPERVISOR. CONTRARY TO THE ABOVE, ON JANUARY 3, 1980, WHILE PERFORMING PROCEDURE C1.3 FOR UNIT 2 SHUTDOWN, AN OPERATOR ERRONEOUSLY CLOSED VALVES CA-1-2 AND CA-1-4 ON UNIT 1, ISOLATING THE CONTAINMENT SPRAY ADDITIVE TANK FROM THE CONTAINMENT SPRAY SYSTEM WHILE UNIT 1 WAS OPERATING, AND DID NOT DELIVER THE SAFEGUARDS HOLD CARDS TO THE SHIFT SUPERVISOR.
(8001 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

Operating routinely.

*LAST IE SITE INSPECTION DATE: January 2-31, 1980

* INSPECTION REPORT NO: 80-01

SUMMARY

INSPECTION SUMMARY

* Inspection on January 2-31, (80-01): Routine, resident inspection of operating events, plant operations, maintenance, security, training, radioactive waste system, and followup of licensee reported events. The inspection involved 138 inspector hours onsite by one NRC inspector. Of the seven areas inspected, no apparent items of noncompliance or deviations were identified in six areas; two apparent items of noncompliance (containment spray additive tank isolated from containment spray system; and failure to follow an operating procedure).

REPORTS RECEIVED FROM LICENSEE

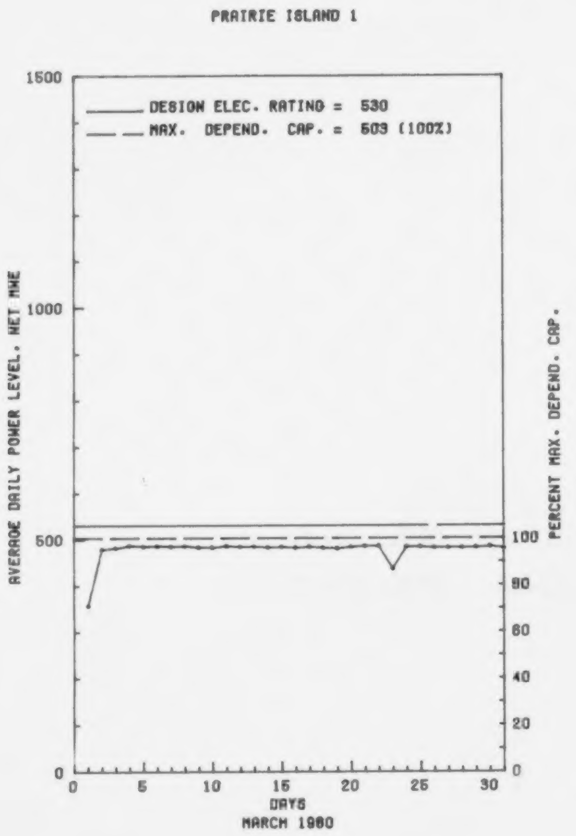
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
80-07/ 03L-0	02/18/80	03/19/80	30 Day LER	During surveillance test the load on D1 Diesel Generator could not be increased above 1400, (full load capability is 2750KW).

REVIEWED BY OMPA LOEB: DATE: 5/12/80

REVIEWED BY NRR DOR: DATE: 5/12/80

PRAIRIE ISLAND 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-282
2. Utility Contact: A. A. Hunstad (612) 388-4767
3. Licensed Thermal Power (MWt): 1650
4. Nameplate Rating (Gross MWe): 659 x 0.9 = 593
5. Design Electrical Rating (Net MWe): 530
6. Maximum Dependable Capacity (Gross MWe): 534
7. Maximum Dependable Capacity (Net MWe): 503
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	2,184.0	55,152.0
12. Hours Reactor Was Critical	744.0	2,052.9	43,261.3
13. Reactor Reserve Shutdown Hours	.0	.0	5,531.9
14. Hours Generator On-Line	744.0	2,047.8	42,171.4
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,208,785	3,328,439	65,407,433
17. Gross Electrical Energy (MWH)	378,650	1,048,490	21,202,160
18. Net Electrical Energy (MWH)	355,732	983,508	19,822,760
19. Unit Service Factor	100.0	93.8	76.5
20. Unit Availability Factor	100.0	93.8	76.5
21. Unit Capacity Factor (MDC Net)	95.1	89.5	71.5
22. Unit Capacity Factor (DER Net)	90.2	85.0	67.8
23. Unit Forced Outage Rate	.0	.0	13.0

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUELING, SUMMER 1980, 5 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrance
	NONE								

SUMMARY

OPERATION WAS BASE-LOADED AT 100% POWER THROUGHOUT THE PERIOD, WITH NO OUTAGES OR REDUCTIONS REPORTED.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

PRAIRIE ISLAND 2

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Minnesota/Goodhue 28 miles SE of Minneapolis
2.	Electric Reliability Council	Mid-Continent Area Reliability Coordination Agreement
3.	Type of Reactor	PWR
4.	Power Level (MWT)	1650
5.	Design Electrical Rating (MWE Net)	530
6.	Date of Initial Criticality	December 17, 1974
7.	Date of Commercial Operation	December 21, 1974
8.	Condenser Cooling Method	Cooling Towers
9.	Condenser Cooling Water Source	Mississippi River

Utility & Contractor Information

10.	Licensee	Northern States Power Company
11.	Corporate Address	414 Nicollet Mall - 8th Floor Minneapolis, Minnesota 55401
12.	Corporate Contact	G. H. Neils (Oper.) L. O. Mayer (Lic.)
13.	Architect/Engineer	Pioneer Services & Eng. Co.
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Northern States Power Co.

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Senior Inspector	R. F. Warnick
19.	IE Resident Inspector	C. D. Feierabend 612-388-8209
20.	Licensing Project Manager	R. Martin
21.	Docket Number	50-306
22.	License Number/Date of Issue	DPR-60 issued 10/29/74
23.	Public Document Room	Environmental Conservation Library Minneapolis Public Library 300 Nicollet Mall Minneapolis, Minnesota 55401

INSPECTION STATUS

ENFORCEMENT STATUS

TECHNICAL SPECIFICATION 6.5 STATES THAT DETAILED WRITTEN PROCEDURES, INCLUDING THE APPLICABLE CHECKOFF LISTS AND INSTRUCTIONS, COVERING AREAS LISTED BELOW SHALL BE PREPARED AND FOLLOWED. THESE PROCEDURES AND CHANGES THERETO, EXCEPT AS SPECIFIED IN TS 6.5.D, SHALL BE REVIEWED BY THE OPERATIONS COMMITTEE AND APPROVED BY A MEMBER OF PLANT MANAGEMENT DESIGNATED BY THE PLANT MANAGER. A. PLANT OPERATIONS (1) INTEGRATED AND SYSTEM PROCEDURES FOR NORMAL STARTUP, OPERATION AND SHUTDOWN OF THE REACTOR AND ALL SYSTEMS AND COMPONENTS INVOLVING NUCLEAR SAFETY OF THE FACILITY. C1.3, UNIT SHUTDOWN PROCEDURE, CONTROLS ACTIONS FOR UNIT SHUTDOWN. (COMPONENT NUMBERS IDENTIFIED BY BRACKETS IDENTIFY UNIT 2 COMPONENTS). STEP 37 OF PROCEDURE C1.3 STATES IN PART: WHEN THE RC TEMPERATURE HAS BEEN REDUCED TO LESS THAN 200 DEGREES F (180 DEGREES F IF PRESSURIZER STEAM BUBBLE IS RETAINED): (A) PLACE NO. 11 AND 12 (21 AND 22) CONTAINMENT SPRAY PUMP CS IN PULLOUT-STOP; AND (B) REMOVE SAFEGUARD HOLD CARD, CLOSE AND ATTACH SECURE CARDS TO CA-1-2 (2CA-1-2), CV-31941, (CV-31939) OUTLET ISOL AND CA-1-4 (2CA-1-4), CV-31938 (CV-31940) OUTLET ISOL. DELIVER SAFEGUARDS HOLD CARD TO THE SHIFT SUPERVISOR. CONTRARY TO THE ABOVE, ON JANUARY 3, 1980, WHILE PERFORMING PROCEDURE C1.3 FOR UNIT 2 SHUTDOWN, AN OPERATOR ERRONEOUSLY CLOSED VALVES CA-1-2 AND CA-1-4 ON UNIT 1, ISOLATING THE CONTAINMENT SPRAY ADDITIVE TANK FROM THE CONTAINMENT SPRAY SYSTEM WHILE UNIT 1 WAS OPERATING, AND DID NOT DELIVER THE SAFEGUARDS HOLD CARDS TO THE SHIFT SUPERVISOR.
(8001 INFRACTION)

TECHNICAL SPECIFICATION 4.2.A.4 STATES IN PART, THAT, NONDESTRUCTIVE INSPECTIONS LISTED IN TABLE TS 4.2-1, SHALL BE PERFORMED AS SPECIFIED. NORTHERN STATES POWER (NSP) PROCEDURE NO. NSP-PT-1, REVISION 1 , SECTION F-1 STATES, "PENETRANT MANUFACTURES CORRESPONDING DEVELOPER, AS LISTED IN FIGURE 1, SHALL BE APPLIED BY SPRAYING IMMEDIATEDLY AFTER THE PENETRANT REMOVAL AND DRYING OPERATION HAS BEEN COMPLETED." THIS PROCEDURE REQUIRES A (5) MINUTE DRYING TIME. ASME SECTION V, ARTICLE 6, SUBARTICLE T-646 STATES, "THE DEVELOPER SHALL BE APPLIED AS SOON AS POSSIBLE AFTER PENETRANT REMOVAL..." CONTRARY TO THE ABOVE, THE INSPECTOR OBSERVED ON JANUARY 17, 1980, DURING A LIQUID PENETRANT EXAMINATION OF PRESSURIZER NOZZLES NO. 156/2 AND NO. 164/2, LINE NO. 6-2RC-20A AND NO. 6-2RC-20B, NAVCO DRAWING NO. A7636(B), THAT DEVELOPER HAD NOT BEEN APPLIED AFTER PENETRANT REMOVAL UNTIL APPROXIMATELY 30 MINUTES ON ONE NOZZLE AND OVER TEN MINUTES ON THE OTHER NOZZLE.
(8003 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENTS

+ Indications of degradation of cold leg steam generator tubes at the first and second support plates were identified during inservice eddy current testing. A degraded tube was removed for destructive testing. (80-01)

FACILITY ITEMS

+ Plant operating routinely.

PLANT STATUS

+ Operating routinely.

+LAST IE SITE INSPECTION DATE: January 2-31, 1980

+ INSPECTION REPORT NO: 80-01

SUMMARY

INSPECTION SUMMARY

+ Inspection on January 2-31, (80-01): Routine, resident inspection of operating events, plant operations, maintenance, security, training, radioactive waste system, and followup of licensee reported events. The inspection involved 138 inspector hours onsite by one NRC inspector. Of the seven areas inspected, no apparent items of noncompliance or deviations were identified in six areas; one apparent items of noncompliance (failure to follow an operating procedure).

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
80-03/ 01X-1	01/09/80 update	03/06/80 03/10/80		During eddy current testing of tubes in Unit 2 steam generators, several indications of tube degradation were seen.
80-08/ 03L-0	02/19/80	03/20/80	30 Day LER	During startup after refueling, one steam flow meter on each steam line did not come on scale.

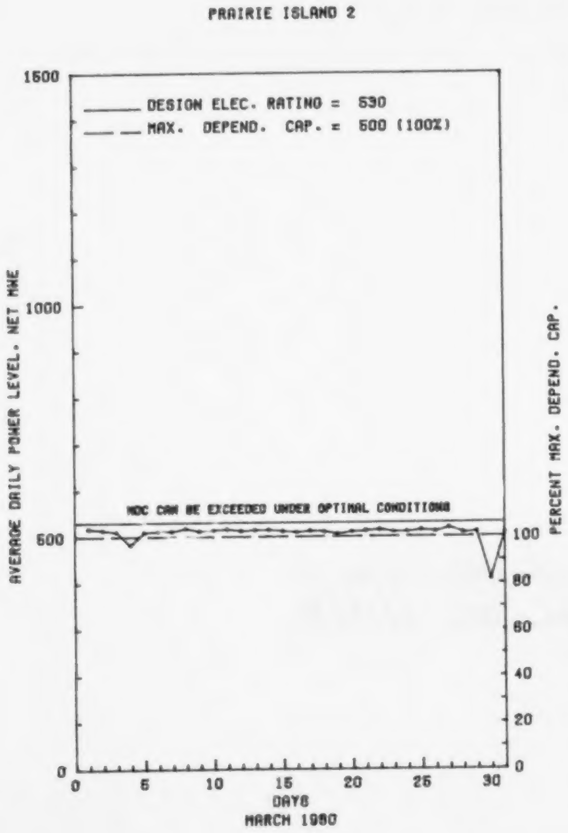
==

REVIEWED BY OMPA LOEB: John R Crooks **DATE:** 5/12/80

REVIEWED BY NRR DOR: DM Grotenhuis **DATE:** 5/12/80

PRAIRIE ISLAND 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-306
2. Utility Contact: A. A. Hunstad (612) 388-4767
3. Licensed Thermal Power (MWt): 1650
4. Nameplate Rating (Gross MWe): 659 x 0.9 = 593
5. Design Electrical Rating (Net MWe): 530
6. Maximum Dependable Capacity (Gross MWe): 531
7. Maximum Dependable Capacity (Net MWe): 500
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	2,184.0	46,270.0
12. Hours Reactor Was Critical	744.0	1,055.3	40,184.3
13. Reactor Reserve Shutdown Hours	.0	.0	1,406.0
14. Hours Generator On-Line	744.0	1,013.5	39,414.5
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,216,287	1,550,737	61,223,240
17. Gross Electrical Energy (MWH)	400,790	510,400	19,564,530
18. Net Electrical Energy (MWH)	377,823	473,220	18,311,852
19. Unit Service Factor	100.0	46.4	85.2
20. Unit Availability Factor	100.0	46.4	85.2
21. Unit Capacity Factor (MDC Net)	101.6	43.3	79.2
22. Unit Capacity Factor (DER Net)	95.8	40.9	74.7
23. Unit Forced Outage Rate	.0	1.0	4.7

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUELING, FEBRUARY 1981, 5 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	NONE								

SUMMARY

OPERATION WAS BASE-LOADED AT 100% POWER THROUGHOUT THE PERIOD, WITH NO OUTAGES OR REDUCTIONS REPORTED.

LEGEND

[1]Type:
F-Forced
S-Scheduled

[2]Reason:
A-Equipment Failure
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error
H-Other

[3]Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Continuations
5-Load Reduction
9-Other

[4]Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5]Exhibit H: Same source.

QUAD CITIES 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Illinois/Rock Island 20 miles NE of Moline
2.	Electric Reliability Council	Mid-America Interpool Network
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2511
5.	Design Electrical Rating (MWE Net)	789
6.	Date of Initial Criticality	October 18, 1971
7.	Date of Commercial Operation	February 18, 1973
8.	Condenser Cooling Method	Spray Canal
9.	Condenser Cooling Water Source	Mississippi River

Utility & Contractor Information

10.	Licensee	Commonwealth Edison Co.
11.	Corporate Address	P. O. Box 767 Chicago, Illinois 60690
12.	Corporate Contact	D. L. Peoples
13.	Architect/Engineer	Sargent and Lundy
14.	Nuclear Steam Supply System	GE
15.	Constructor	United Engineers & Constructors, Inc.

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Senior Inspector	R. Spessard
19.	IE Resident Inspector	N. J. Chrissotimus (309) 654-2227
20.	Licensing Project Manager	R. B. Bevan
21.	Docket Number	50-254
22.	License Number/Date of Issue	DPR-29 issued 10/1/71
23.	Public Document Room	Moline Public Library 504 17th Street Moline, Illinois 61265

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

MANAGERIAL ITEMS

+ A corporate management meeting was held to discuss recent regulatory performance of the station. (80-01)

PLANT STATUS

Unit operating routinely.

+LAST IE SITE INSPECTION DATE: January 15-16 and February 11, 1980

+ INSPECTION REPORT NO: 80-03

SUMMARY

INSPECTION SUMMARY

+ Inspection on January 2 through February 1, (80-01): Routine, unannounced resident inspection of licensee actions on previous inspection findings, maintenance, plant operations, physical protection-security organization; physical barriers; access control (identification, authorization, badging, search and escorting); and communications, review and followup on licensee event reports, review and followup on IE Bulletins, surveillance of core power distribution limits, and thermal power evaluation. The inspection involved 175 inspector hours onsite by one NRC inspector. Of the 12 areas inspected, no apparent items of noncompliance were identified.

+ Inspection on January 15-16 and February 11, (80-03): Torus support modification QA/QC program, implementing procedures, work activities, and review and evaluation of welding and nondestructive examination (NDE) data and reports; interpret radiographs of welds in feedwater system. The inspection involved 19 inspector hours onsite by one NRC inspector. No items of noncompliance or deviations were identified.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
80-03/ 03L-0	02/06/80	03/06/80	30 Day LER	Circuit breaker tripped out while performing QOS 1000-3 LPCI Motor Operated Valve Operability Test.
80-04/ 03L-0	02/15/80	02/26/80	30 Day LER	QIS-11, level switch LIS-1-263-72A failed to operate properly.
80-06/ 03L-0	02/22/80	03/14/80	30 Day LER	The RCIC Steam Supply valves MO 1-1301 16 and 17 were found in the closed position.

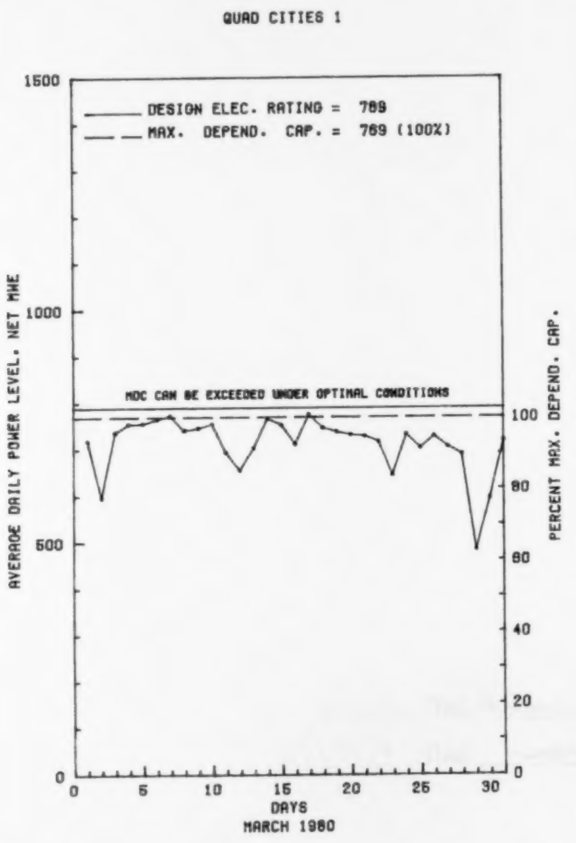
==

REVIEWED BY OMPA LOEB: DATE: 5/12/80

REVIEWED BY NRR DOR: RBBevan DATE: 5/13/80

QUAD CITIES 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-254
2. Utility Contact: D. Hannum (309) 654-2241
3. Licensed Thermal Power (MWt): 2511
4. Nameplate Rating (Gross MWe): 920 x 0.9 = 828
5. Design Electrical Rating (Net MWe): 789
6. Maximum Dependable Capacity (Gross MWe): 813
7. Maximum Dependable Capacity (Net MWe): 769
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	2,184.0	69,144.0
12. Hours Reactor Was Critical	744.0	2,184.0	56,898.1
13. Reactor Reserve Shutdown Hours	.0	.0	3,421.9
14. Hours Generator On-Line	744.0	2,184.0	54,224.9
15. Unit Reserve Shutdown Hours	.0	.0	909.2
16. Gross Thermal Energy (MWH)	1,709,686	4,989,236	109,533,217
17. Gross Electrical Energy (MWH)	560,570	1,637,457	35,260,511
18. Net Electrical Energy (MWH)	529,713	1,556,629	32,972,165
19. Unit Service Factor	100.0	100.0	78.4
20. Unit Availability Factor	100.0	100.0	79.7
21. Unit Capacity Factor (MDC Net)	92.6	92.7	62.0
22. Unit Capacity Factor (DER Net)	90.2	90.3	60.4
23. Unit Forced Outage Rate	.0	.0	7.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
4	03/02/80	S	.0	B	5				UNIT ONE LOAD WAS REDUCED FOR A CONTROL ROD PATTERN CHANGE.
5	03/12/80	S	.0	B	5				UNIT ONE LOAD WAS REDUCED FOR A CONTROL ROD PATTERN CHANGE AND TO CHANGE OVER CONDENSATE DEMINERALIZERS.

SUMMARY

OPERATION WAS ROUTINE DURING THE REPORT PERIOD, WITH BRIEF LOAD REDUCTIONS FOR CONTROL ROD ADJUSTMENT AND DUE TO HIGH DIFFERENTIAL PRESSURE IN CONDENSATE DEMINERALIZER SYSTEM.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

QUAD CITIES 2

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Illinois/Rock Island 20 miles NE of Moline
2.	Electric Reliability Council	Mid-America Interpool Network
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2511
5.	Design Electrical Rating (MWE Net)	789
6.	Date of Initial Criticality	April 26, 1972
7.	Date of Commercial Operation	March 10, 1973
8.	Condenser Cooling Method	Spray Canal
9.	Condenser Cooling Water Source	Mississippi River

Utility & Contractor Information

10.	Licensee	Commonwealth Edison Co.
11.	Corporate Address	P. O. Box 767 Chicago, Illinois 60690
12.	Corporate Contact	D. L. Peoples
13.	Architect/Engineer	Sargent & Lundy
14.	Nuclear Steam Supply System	GE
15.	Constructor	United Engineers & Constructors, Inc.

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Senior Inspector	R. Spessard
19.	IE Resident Inspector	N. J. Chrissotimus (309) 654-2227
20.	Licensing Project Manager	R. B. Bevan
21.	Docket Number	50-265
22.	License Number/Date of Issue	DPR-30 issued 3/21/72
23.	Public Document Room	Moline Public Library 504 17th Street Moline, Illinois 61265

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

FACILITY ITEMS

+ Unit 2 refueling/maintenance outage is expected to last until April, 1980. Torus modification and clad removal of feedwater sparger nozzles is progressing. (80-02)

MANAGERIAL ITEMS

+ A corporate management meeting was held to discuss recent regulatory performance of the station. (80-02)

PLANT STATUS

+ The unit was shut down on November 25, 1979 to begin a scheduled 13-week refueling and maintenance outage. The outage work is completed and the unit returned to service on April 20, 1980.

+LAST IE SITE INSPECTION DATE: January 30-31, 1980

+ INSPECTION REPORT NO: 80-07

SUMMARY

INSPECTION SUMMARY

+ Inspection on January 2 through February 1, (80-02): Routine, unannounced resident inspection of licensee actions on previous inspection findings, maintenance, plant operations, physical protection-security organization; physical barriers; access control (identification, authorization, badging, search and escorting); and communications, review and followup on licensee event reports, review and followup on IE Bulletins, surveillance of core power distribution limits, and thermal power evaluation. The inspection involved 175 inspector hours onsite by one NRC inspector. Of the 12 areas inspected, no apparent items of noncompliance were identified.

+ Inspection on January 15-16 and February 11, (80-05): Torus support modification QA/QC program, implementing procedures, work activities, and review and evaluation of welding and nondestructive examination (NDE) data and reports; interpret radiographs of welds in feedwater system. The inspection involved 19 inspector hours onsite by one NRC inspector. No items of noncompliance or deviations were identified.

+ Inspection on January 30-31, (80-07): IE Bulletin 80-02, Inadequate Quality Assurance for Nuclear Supplied Equipment; examination of General Electric (GE) supplied replacement feedwater spargers and review of documentation. The inspection involved 12 inspector hours onsite by one NRC inspector. No items of noncompliance or deviations were identified.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

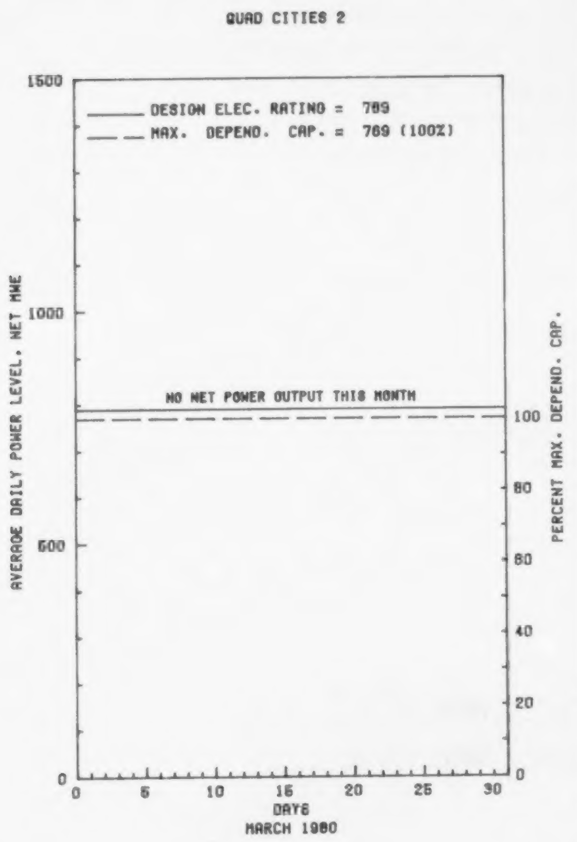
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
80-04/03L-0	02/19/80	03/06/80	30 Day LER	Auto. blowdown timer 287-150A took 122 seconds to time out, in excess of TS.

REVIEWED BY OMPA LOEB: DATE: 5/12/80

REVIEWED BY NRR DOR: RBBevan DATE: 5/13/80

QUAD CITIES 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-265
2. Utility Contact: D. Hannum (309) 654-2241
3. Licensed Thermal Power (MWt): 2511
4. Nameplate Rating (Gross MWe): 920 x 0.9 = 828
5. Design Electrical Rating (Net MWe): 789
6. Maximum Dependable Capacity (Gross MWe): 813
7. Maximum Dependable Capacity (Net MWe): 769
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	2,184.0	68,254.0
12. Hours Reactor Was Critical	.0	.0	53,128.9
13. Reactor Reserve Shutdown Hours	.0	.0	2,985.8
14. Hours Generator On-Line	.0	.0	50,791.8
15. Unit Reserve Shutdown Hours	.0	.0	702.9
16. Gross Thermal Energy (MWH)	0	0	103,016,109
17. Gross Electrical Energy (MWH)	22	49	32,863,099
18. Net Electrical Energy (MWH)	-10,581	-37,666	30,704,859
19. Unit Service Factor	.0	.0	74.4
20. Unit Availability Factor	.0	.0	75.4
21. Unit Capacity Factor (MDC Net)	.0	.0	58.5
22. Unit Capacity Factor (DER Net)	.0	.0	57.0
23. Unit Forced Outage Rate	.0	.0	9.3

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 04/21/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
13	11/25/79	S	744.0	C	4		RC	FUELXX	UNIT TWO CYCLE 4 REFUELING OUTAGE CONTINUES.

SUMMARY

THE UNIT REMAINED SHUTDOWN THROUGHOUT THE REPORT PERIOD FOR REFUELING.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

RANCHO SECO

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	California/Sacramento 25 miles SE of Sacramento
2.	Electric Reliability Council	Western Systems Coordinating Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2772
5.	Design Electrical Rating (MWE Net)	918
6.	Date of Initial Criticality	September 16, 1974
7.	Date of Commercial Operation	April 17, 1975 (92.6% of full power)
8.	Condenser Cooling Method	Closed Cycle
9.	Condenser Cooling Water Source	Natural Draft Cooling Towers - Folsom Canal

Utility & Contractor Information

10.	Licensee	Sacramento Municipal Utility District
11.	Corporate Address	6201 South Street P. O. Box 15830 Sacramento, California
12.	Corporate Contact	J. J. Mattimoe Assistant General Manager, Chief Engineer
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	B&W
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	V
17.	IE Branch Chief	J. L. Crews
18.	IE Senior Inspector	B. H. Faulkenberry
19.	IE Resident Inspector	H. L. Canter (209) 748-2791
20.	Licensing Project Manager	D. J. Garner
21.	Docket Number	50-312
22.	License Number/Date of Issue	DPR-54, 8/16/74
23.	Public Document Room	Business and Municipal Department Sacramento City-County Library 828 I Street Sacramento, California 95814

INSPECTION STATUS

ENFORCEMENT STATUS

TS 6.9.4.1.N REQUIRES A PROMPT REPORT WHEN STRUCTURES, SYSTEMS, OR COMPONENTS REQUIRE REMEDIAL ACTION OR CORRECTIVE MEASURES TO PREVENT OPERATION IN A LESS SAFE CONDITION THAN ANALYZED. CONTRARY TO THIS, A SIGNIFICANT PROBLEM WITH THE CONTAINMENT PURGE AND VENT VALVES WAS NOTED BY COGNIZANT LICENSEE PERSONNEL ON DECEMBER 18, 1979 BUT WAS NOT REPORTED TO THE NRC UNTIL JANUARY 8, 1980. (8003 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST (CITE RO REPORT NUMBER, IF APP.)

SYSTEMS AND COMPONENT PROBLEMS:

None.

FACILITY ITEMS (PLANS AND PROCEDURES):

None.

MANAGERIAL ITEMS:

None.

PLANT STATUS:

Plant refueling outage.

LAST IE SITE INSPECTION DATE: 04/07-11/80+

INSPECTION REPORT NO. 50-312/80-09+

SUMMARY

INSPECTION SUMMARY

+ Inspection on January 17-26, 1980 (Report No. 50-312/80-01)
Areas Inspected: Routine, unannounced inspection of progress in completion of required fire protection modifications, major surveillance performed during a refueling outage; major maintenance performed during a refueling outage; followup of unresolved items; and followup of IE Bulletins and Circulars. The inspection involved 64 inspector hours on site by one NRC inspector.

Results: No items of noncompliance or deviations were identified.

+ Inspection on February 12-15, 28-29 & March 3-7, 1980 (Report No. 50-312/80-04)
Areas Inspected: Routine, unannounced inspection of the status of completion of selected fire protection modifications; status of completion of other required modifications; maintenance during a refueling outage; followup on IE Bulletins; and followup on previous inspection findings. The inspection involved 65 inspector hours on site by one NRC inspector.

Results: No items of noncompliance or deviations were identified.

+ Inspection on February 1-29, 1980 (Report No. 50-312/80-06)
Areas Inspected: Routine inspections of preparation for refueling activities; refueling activities; LER followup; facility modifications; and independent inspection effort. The inspection involved 100 inspector hours by three NRC Region V based inspectors.

Results: No items of noncompliance were identified in four areas. Three items of noncompliance were identified in one area (LER followup). Assessment of a civil penalty in the amount of $25,000 is proposed.

+ Inspection on March 17-21, 1980 (Report No. 50-312/80-07)
Inspection cancelled.

+ Inspection on March 4-5, 1980 (Report No. 50-312/80-08)
Areas Inspected: Routine, unannounced inspection by Regional based inspectors of the licensee's activities performed in response to IE Bulletin 79-02, "Pipe Support Base Plate Design Using Concrete Expansion Anchor Bolts." The inspection involved 16 inspector hours onsite by one NRC inspector.

Results: No items of noncompliance of deviations were identified.

+ Inspection on April 7-11, 1980 (Report No. 50-312/80-09)
Report being prepared; to be reported next month.

+ Inspection on March 1-31, 1980 (Report No. 50-312/80-10)
Report being prepared; to be reported next month.

NRC POWER RESTRICTIONS: None

REPORTS RECEIVED FROM LICENSEE

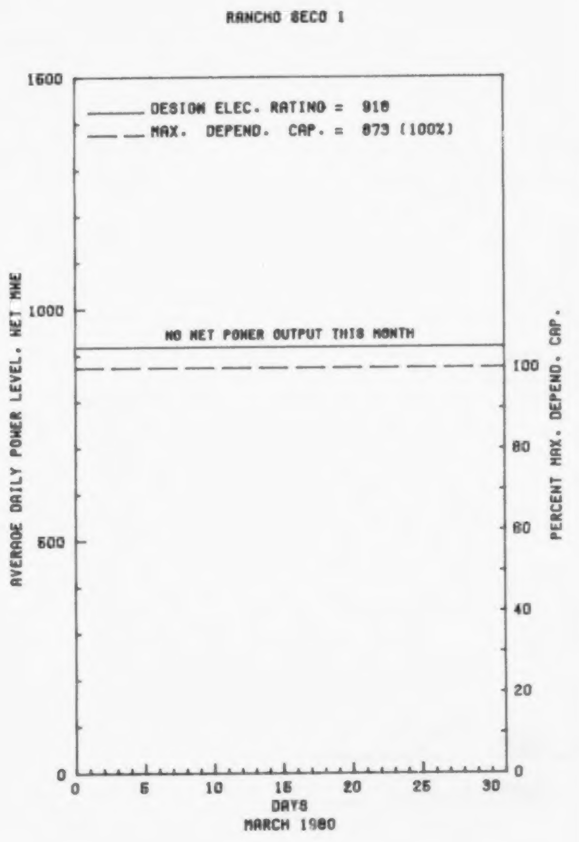
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-20/ 03L-1	12-07-79	02-26-80	30 Day LER	Socket weld leak - Will not replace valve in 1980 outage.
80-07/ 03L-0	01-25-80	02-25-80	30 Day LER	Seven Bergen-Paterson hydraulic snubbers failed functional tests.
80-08/ 03L-0	02-03-80	02-26-80	30 Day LER	U-bolt pipe support missing on 3/4 inch line 26085-3/4-EDI.
80-09/ 03L-0	02-07-80	02-28-80	30 Day LER	Auto reclose contacts in safety breakers not wired and will not be.
80-10/ 03L-0	02-13-80	02-28-80	30 Day LER	Seven ton cask handled over fuel assemblies - Violation of TS 3.11.
80-11/ 03L-0	02-19-80	03-11-80	30 Day LER	OTSG Inspection Results: Tube 75-15 in A and 60-43 and 82-12 in B plugged.
80-12/ 03L-0	02-19-80	03-06-80	30 Day LER	Diesel generator A failed to start due to metal debris in manual start relay.
80-13/ 03L-0	02-18-80	03-07-80	30 Day LER	DSL generator B DC starting system inoperable due to leak in air line.
80-14/ 03L-0	02-19-80	03-11-80	30 Day LER	Failure of wire rope sling causing aris calib block to fall on seal.

REVIEWED BY OMPA LOEB: E Bobe **DATE:** 5/12/80

REVIEWED BY NRR DOR: DJGarner **DATE:** 5/12/80

RANCHO SECO

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-312
2. Utility Contact: R. W. Colombo (916) 452-3211
3. Licensed Thermal Power (MWt): 2772
4. Nameplate Rating (Gross MWe): 1070 x 0.9 = 963
5. Design Electrical Rating (Net MWe): 918
6. Maximum Dependable Capacity (Gross MWe): 917
7. Maximum Dependable Capacity (Net MWe): 873
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	2,184.0	43,441.0
12. Hours Reactor Was Critical	.0	271.0	26,344.5
13. Reactor Reserve Shutdown Hours	.0	.0	3,975.1
14. Hours Generator On-Line	.0	266.9	25,208.2
15. Unit Reserve Shutdown Hours	.0	.0	1,210.2
16. Gross Thermal Energy (MWH)	0	689,820	63,673,838
17. Gross Electrical Energy (MWH)	0	221,784	21,506,320
18. Net Electrical Energy (MWH)	0	206,442	20,317,332
19. Unit Service Factor	.0	12.2	58.0
20. Unit Availability Factor	.0	12.2	60.8
21. Unit Capacity Factor (MDC Net)	.0	10.8	53.6
22. Unit Capacity Factor (DER Net)	.0	10.3	50.9
23. Unit Forced Outage Rate	.0	14.5	32.0

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 04/15/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
02	01/19/80	S	744.0	C	4				3RD REFUELING OUTAGE CONTINUES.

SUMMARY

THE UNIT REMAINED SHUTDOWN THROUGHOUT THE REPORT PERIOD FOR REFUELING.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

ROBINSON 2

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	South Carolina/Darlington 5miles NW of Hartsville
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2300
5.	Design Electrical Rating (MWE Net)	700
6.	Date of Initial Criticality	September 20, 1970
7.	Date of Commercial Operation	March 7, 1971
8.	Condenser Cooling Method	Recirculation from impounded lake
9.	Condenser Cooling Water Source	Robinson Impoundment

Utility & Contractor Information

10.	Licensee	Carolina Power and Light Company
11.	Corporate Address	411 Fayetteville Street Raleigh, North Carolina
12.	Corporate Contact	J. A. Jones Executive Vice President
13.	Architect/Engineer	Ebasco
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Ebasco

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R.C. Lewis
18.	IE Senior Inspector	P. J. Kellogg
19.	IE Principal Inspector	R. Sauer
20.	Licensing Project Manager	D. Neighbors
21.	Docket Number	50-261
22.	License Number/Date of Issue	DPR-23, 7/31/70 (5 Mwt), Full power authorization issued 9/23/70
23.	Public Document Room	Hartsville Memorial Library Home and Fifth Avenue Hartsville, South Carolina 29550

INSPECTION STATUS

ENFORCEMENT STATUS

AS REQUIRED BY TECHNICAL SPECIFICATION 6.8.1, WRITTEN PROCEDURES SHALL BE ESTABLISHED, IMPLEMENTED AND MAINTAINED THAT MEET OR EXCEED THE REQUIREMENTS AND RECOMMENDATIONS OF SECTIONS 5.1 AND 5.3 OF ANSI N18.7-1972 AND APPENDIX "A" OF USNRC REGULATORY GUIDE 1.33 DATED NOVEMBER 3, 1972 ANSI N18.7-1972, SECTION 5.3.7, STATES IN PART THAT PROCEDURES SHALL BE PROVIDED FOR THE CONTROL, TREATMENT, AND MANAGEMENT OF RADIOACTIVE WASTES. R.G. 1.33 (1972).APPENDIX A, SECTION G, INCLUDES SPENT RESIN AND FILTER SLUDGE HANDLING, BALING MACHINE OPERATION, AND DRUM HANDLING AND STORAGE AS TYPICAL SAFETY-RELATED ACTIVITIES WHICH SHOULD BE COVERED BY WRITTEN PROCEDURES. THERE WERE NO LICENSEE-APPROVED WRITTEN PROCEDURES FOR THE METHODS AND TECHNIQUES ROUTINELY USED TO TRANSFER AND DEWATER SPENT RESIN OR TO SOLIDIFY EVAPORATOR BOTTOMS.
(7922 INFRACTION)

AS REQUIRED BY TECHNICAL SPECIFICATION 6.8.1, WRITTEN PROCEDURES SHALL BE ESTABLISHED, IMPLEMENTED AND MAINTAINED THAT MEET OR EXCEED THE REQUIREMENTS AND RECOMMENDATIONS OF SECTION 5.1 AND 5.3 OF ANSI N18.7-1972 AND APPENDIX A OF USNRC REGULATORY GUIDE 1.33 DATED NOVEMBER 3, 1972. REGULATORY GUIDE 1.33 STATES IN PART THAT CHEMICAL AND RADIOCHEMICAL CONTROL PROCEDURES SHOULD BE WRITTEN TO PRESCRIBE THE NATURE AND FREQUENCY OF SAMPLING AND ANALYSES AND THAT THESE PROCEDURES SHOULD SPECIFY LABORATORY INSTRUCTIONS AND CALIBRATION OF LABORATORY EQUIPMENT. ON FEBRUARY 5, 1980 THERE WERE NO APPROVED WRITTEN PROCEDURES FOR OPERATION OF THE GE(LI) GAMMA RAY SPECTROSCOPY SYSTEM AND APPROVED WRITTEN PROCEDURES WERE NOT USED FOR RADIOCHEMICAL ANALYSIS OF SAMPLES FOR MONITORING EFFLUENT RELEASES.
(8001 INFRACTION)

FAILURE TO MAINTAIN PROTECTED AREA AS REQUIRED BY SECURITY PLAN.
(8002 INFRACTION)

FAILURE TO SEARCH VISITORS AS REQUIRED BY SECURITY PLAN.
(8002 INFRACTION)

CONTRARY TO THE REQUIREMENTS OF 10CFR20.203(F), CONTAINERS CALCULATED TO CONTAIN LICENSED MATERIAL IN THE CONTAMINATED WAREHOUSE AND IN 55-GALLON DRUMS OUTSIDE THE BIT ROOM WERE NOT LABELED PROPERLY.
(8003 DEFICIENCY)

CONTRARY TO THE REQUIREMENTS OF T.S. 6.11, HP-20 WAS NOT ADHERED TO IN FULL IN THAT ITEM 3 IN ANSWER TO QUESTION 0 REQUIRING COMPLIANCE WITH CNSI RADIOACTIVE MATERIAL LICENSES WAS NOT MET. CONDITION 29 OF CNSI LICENSE 097 (AMENDMENT 26) STATES THAT WASTE CONTAINING BOTH TOXI CHEMICALS AND RADIOACTIVE MATERIALS SHALL REQUIRE AN INDEPENDENT EVALUATION OF BOTH HAZARDS. ON FEBRUARY 5, 1980, AN LSA CONTAINER PREPARED FOR SHIPMENT TO BARNWELL CONTAINED SULFURIC ACID WITHOUT ANY PREVIOUS EVALUATION OF THE CHEMICAL VERSUS RADIOACTIVE HAZARDS.
(8003 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

UNIT IS IN NORMAL OPERATION.

* LAST IE SITE INSPECTION DATE: MARCH 31 - APRIL 3, 1980

* INSPECTION REPORT NO: 50-261/80-07

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
80-02/ 03L-0	01/27/80	02/25/80	30 DAY LER	ENGINE DRIVEN FIRE PUMP FAILED DUE TO DRIVE SHAFT SEPARATION.

SUMMARY

INSPECTION SUMMARY

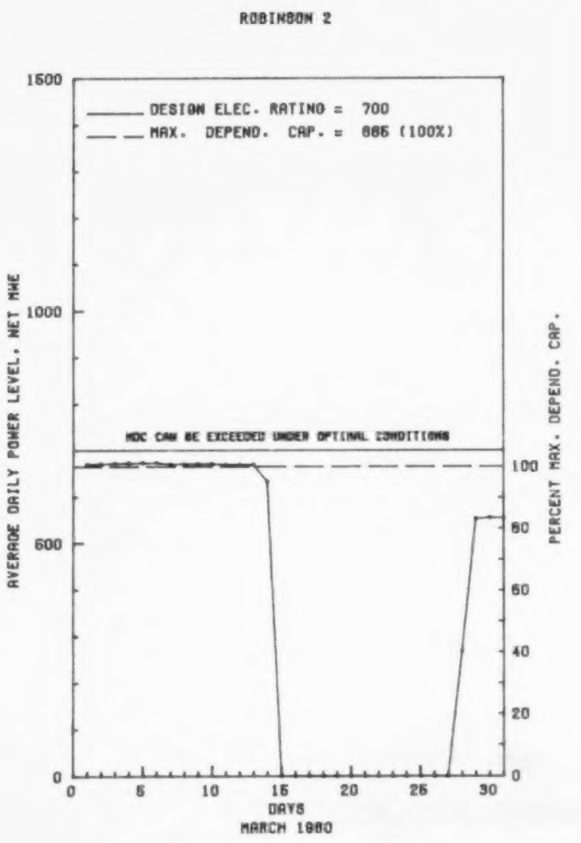
* INSPECTION FEBRUARY 20-21 (80-05): THIS SPECIAL, ANNOUNCED INSPECTION INVOLVED 13 INSPECTOR-HOURS ON SITE IN THE AREAS OF SEISMIC ANALYSIS FOR AS-BUILT SAFETY-RELATED PIPING SYSTEMS (IE BULLETIN 79-14). NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION MARCH 10-13 (80-08): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 22 INSPECTOR-HOURS ON SITE IN THE AREAS OF OPERATIONS; LICENSEE EVENT REPORTS; CIRCULARS AND BULLETINS AND OPEN ITEMS. OF THE FOUR AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

REVIEWED BY OMPA LOEB: RE Bobe **DATE:** 5/12/80
REVIEWED BY NRR DOR: Don Neighbors **DATE:** 5/12/80

ROBINSON 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-261
2. Utility Contact: M. L. Watford (803) 383-4524
3. Licensed Thermal Power (MWt): 2300
4. Nameplate Rating (Gross MWe): 854 x 0.9 = 769
5. Design Electrical Rating (Net MWe): 700
6. Maximum Dependable Capacity (Gross MWe): 700
7. Maximum Dependable Capacity (Net MWe): 665
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): 2200
10. Reasons for Restrictions, If Any:
EXCESSIVE MOISTURE CARRY-OVER TO H.P. TURBINE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	2,184.0	79,566.0
12. Hours Reactor Was Critical	426.8	1,866.8	62,073.7
13. Reactor Reserve Shutdown Hours	157.2	157.2	876.5
14. Hours Generator On-Line	422.5	1,862.5	60,553.4
15. Unit Reserve Shutdown Hours	0	0	23.2
16. Gross Thermal Energy (MWH)	875,306	3,945,088	124,495,417
17. Gross Electrical Energy (MWH)	285,058	1,295,428	40,231,941
18. Net Electrical Energy (MWH)	267,406	1,230,019	38,135,421
19. Unit Service Factor	56.8	85.3	76.1
20. Unit Availability Factor	56.8	85.3	76.1
*21. Unit Capacity Factor (MDC Net)	54.0	84.7	72.1
22. Unit Capacity Factor (DER Net)	51.3	80.5	68.5
23. Unit Forced Outage Rate	43.2	14.7	13.2

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
REFUELING/MAINTENANCE, JUNE, 6 WEEKS
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type¹	Duration (Hours)	Reason²	Method of Shutting Down Reactor³	License Event Report No.	System Code⁴	Component Code⁵	Cause & Corrective Action to Prevent Recurrence
*03/01	03/14/80	F	321.5	B	1		CB	HTEXCH	STEAM GENERATOR TUBE LEAKS. EDDY CURRENT TESTING DONE IN "A" SG(39% INSPECTED) AND "C" SG(25% INSPECTED). THREE LEAKING TUBES PLUGGED AND A TOTAL OF 57 TUBES PLUGGED BASED ON E/C TESTING AND RELATED WORK.

SUMMARY

OPERATION WAS AT NEAR FULL POWER UNTIL 03/14/80, WHEN THE UNIT WAS SHUTDOWN THROUGH 03/27/80 TO REPAIR STEAM GENERATOR TUBE LEAKS.

LEGEND

¹Type:
F--Forced
S--Scheduled

²Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

³Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

⁴Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

⁵Exhibit H:
Same source.

SALEM 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	New Jersey/Salem 20 miles S of Wilmington, Del.
2.	Electric Reliability Council	Mid-Atlantic Area Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	3338
5.	Design Electrical Rating (MWE Net)	1090
6.	Date of Initial Criticality	December 11, 1976
7.	Date of Commercial Operation	June 30, 1977
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Delaware River

Utility & Contractor Information

10.	Licensee	Public Service Electric & Gas Co.
11.	Corporate Address	80 Park Place Newark, New Jersey 07101
12.	Corporate Contact	F. P. Librizzi General Manager - Electric Prod.
13.	Architect/Engineer	PSE&G
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	UE&C

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	IE Senior Inspector	R. R. Keimig
19.	IE Resident Inspector	L. J. Norrholm (609-935-3850) (FTS 341-3000)
20.	Licensing Project Manager	W. Ross
21.	Docket Number	50-272
22.	License Number/Date of Issue	DPR-70, 8/13/76
23.	Public Document Room	Salem Free Public Library 112 West Broadway Salem, New Jersey 08079

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO TS 6.8.1, AN INDIVIDUAL NOT AUTHORIZED OR FAMILIAR WITH TS REQUIREMENTS APPROVED A TAGGING REQUEST WITHOUT THE KNOWLEDGE OF THE RESPONSIBLE SHIFT SUPERVISOR THAT RESULTED IN ALL THREE EMERGENCY DIESELS BEING TAGGED & LOCKED OUT INSTANTANEOUSLY.
(7932 INFRACTION)

CONTRARY TO FSAR DESCRIPTION, AUXILIARY FEEDWATER STORAGE TANK AS-BUILT WAS NOT SEISMIC I.
(7933 DEVIATION)

CONTRARY TO TS 6.8.1 & PERFORMANCE DEPARTMENT MANUAL, PARAGRAPHS 3.4.5.5 & 3.4.5.7, QUALITY CONTROL & SUPERVISORY REVIEW DID NOT OBSERVE OR CORRECT IMPROPER RECORDED DATA IN 2 T.S. CALIBRATION PROCEDURES.
(8001 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

Leakage from the spent fuel pool liner of between 500-1000 cc/hr. Repairs are in progress on the suspected leak area.

PLANT STATUS:

Operating at full power.

LAST IE SITE INSPECTION DATE: 3/12/80

INSPECTION REPORT NO: 50-272/80-08

SUMMARY

INSPECTION SUMMARY

50-272/79-32 - Nov 18 - Dec 15, 1979 & Jan 9, 1980: Routine inspections by the resident inspector of plant operations including: tours of the facility; log and record reviews; review of licensee events; IE Bulletins and Circulars; implementation of licensing commitments; and, followup on previous inspection items. The inspections involved 49 hours by the NRC resident inspector and 6 hours by one region-based inspector. Noncompliances: One (failure to follow procedures).

50-272/79-33 - Dec 16, 1979 - Feb 2, 1980: Routine inspections by the NRC resident inspector (40 hours) of plant operations including: tours of the facility; log and record reviews, review of licensee events; IE Bulletins; Immediate Action Letter followup; observations in the control room; and followup on previous inspection items. Noncompliances: None. Deviation: One (change made to the facility as described in the FSAR).

50-272/80-01 - Jan 22-25: Routine, unannounced inspection by a region-based inspector (29 hours) of licensee action on previous inspections findings; administrative control of safety related calibrations; surveillance calibrations of safety related components and equipment required by Technical Specifications; calibrations required by Technical Specifications of components and equipment association with safety related systems and/or functions; calibration and control of test equipment; witnessing of calibrations; technician qualifications; and facility tour. Noncompliances: One (failure to properly review calibration data).

REPORTS RECEIVED FROM LICENSEE

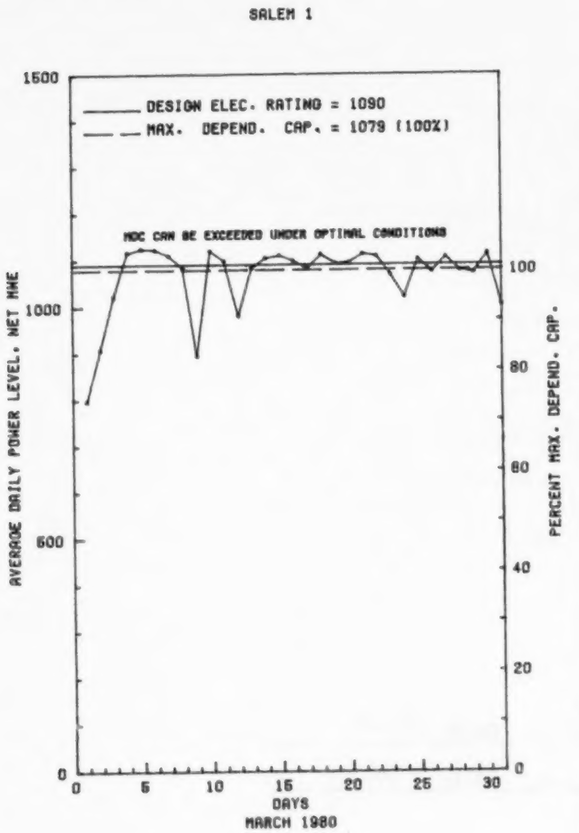
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
	NONE			

REVIEWED BY OMPA LOEB: Bobe **DATE:** 5/12/80

REVIEWED BY NRR DOR: Reeves **DATE:** 5-13-80

SALEM 1

AVERAGE DAILY POWER LEVEL(MWe)



OPERATING STATUS

1. Docket Number: 50-272
2. Utility Contact: L. K. Miller (609) 365-7000
3. Licensed Thermal Power (MWt): 3338
4. Nameplate Rating (Gross MWe): 1300 x 0.9 = 1170
5. Design Electrical Rating (Net MWe): 1090
6. Maximum Dependable Capacity (Gross MWe): 1124
7. Maximum Dependable Capacity (Net MWe): 1079
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	2,184.0	24,145.0
12. Hours Reactor Was Critical	744.0	2,093.5	12,181.8
13. Reactor Reserve Shutdown Hours	.0	.0	22.7
14. Hours Generator On-Line	744.0	1,994.3	11,525.4
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	2,405,018	5,975,448	33,511,744
17. Gross Electrical Energy (MWH)	823,870	2,012,600	11,180,100
18. Net Electrical Energy (MWH)	792,307	1,923,293	10,552,883
19. Unit Service Factor	100.0	91.3	47.7
20. Unit Availability Factor	100.0	91.3	47.7
21. Unit Capacity Factor (MDC Net)	98.7	81.6	40.5
22. Unit Capacity Factor (DER Net)	97.7	80.8	40.1
23. Unit Forced Outage Rate	.0	8.7	40.7

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUELING - 9/20/80
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-062	03/01/80	F	.0	A	5		HH	FILTER	CLEAN COND. PUMP SUCTION STRAINERS
80-064	03/02/80	F	.0	A	5		HG	DEMINX	HIGH DIFFERENTIAL PRESSURE CONDENSATE POLISHERS
80-066	03/03/80	F	.0	A	5		HH	PUMPXX	13A VACUUM PUMP TAGGED OUT FOR REPAIRS
80-068	03/06/80	F	.0	A	5		HH	HTEXCH	CLEAN 13B CONDENSER WATER BOX
80-072	03/09/80	F	.0	A	5		HH	FILTER	CLEAN COND. PUMP SUCTION STRAINERS
80-073	03/13/80	F	.0	A	5		HH	HTEXCH	CLEAN 13A CONDENSER WATER BOX
80-074	03/12/80	F	.0	A	5		HH	FILTER	CLEAN COND. PUMP SUCTION STRAINERS
80-976	03/16/80	F	.0	A	5		HH	HTEXCH	CLEAN 13B CONDENSER WATER BOX
80-078	03/16/80	F	.0	A	5		HH	HTEXCH	CLEAN 13A CONDENSER WATER BOX
80-079	03/18/80	F	.0	A	5		HH	HTEXCH	CLEAN 13B CONDENSER WATER BOX
80-081	03/18/80	F	.0	A	5		HH	HTEXCH	CLEAN 13B CONDENSER WATER BOX
80-083	03/23/80	F	.0	A	5		HH	HTEXCH	CLEAN 11B CONDENSER WATER BOX
80-085	03/23/80	F	.0	A	5		HH	FILTER	CLEAN CONDENSER PIPE STRAINERS
80-086	03/26/80	F	.0	A	5		HH	HTEXCH	CLEAN 13A CONDENSER WATER BOX
80-088	03/31/80	F	.0	A	5		HH	FILTER	CLEAN COND. PUMP SUCTION STRAINERS

SUMMARY

THE UNIT EXPERIENCED SEVERAL BRIEF REDUCTIONS FOR CONDENSER WATERBOX, PIPE STRAINER, AND PUMP SUCTION STRAINER CLEANING.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

SAN ONOFRE 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	California/San Diego 5 miles S of San Clemente
2.	Electric Reliability Council	Western Systems Coordinating Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	1347
5.	Design Electrical Rating (MWE Net)	436
6.	Date of Initial Criticality	June 14, 1967
7.	Date of Commercial Operation	January 1, 1968
8.	Condenser Cooling Method	Circulating sea water
9.	Condenser Cooling Water Source	Pacific Ocean

Utility & Contractor Information

10.	Licensee	Southern California Edison Co.
11.	Corporate Address	2244 Walnut Grove Avenue Rosemead, California 91770
12.	Corporate Contact	R. E. Woodbury V.P. & General Counsel
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	V
17.	IE Branch Chief	J. L. Crews
18.	IE Senior Inspector	B. H. Faulkenberry
19.	IE Resident Inspector	R. Pate (714) 492-2641
20.	Licensing Project Manager	T. Wambach
21.	Docket Number	50-206
22.	License Number/Date of Issue	DPR-13, 3/27/67
23.	Public Document Room	Mission Viejo Branch Library 24851 Chrisanta Drive Mission Viejo, California 92676

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO THE REQUIREMENTS OF 10 CFR 50, APPENDIX B, CRITERION V, THE LICENSEE'S QUALITY ASSURANCE PROGRAM AND IMPLEMENTING PROCEDURE WPP 19.20, WELD FILLER MATERIAL WAS FOUND TO BE IMPROPERLY CONTROLLED. UNAUTHORIZED FILLER MATERIAL, UNPROTECTED LOW HYDROGEN MATERIAL, AND IMPROPERLY DISCARDED FILLER MATERIAL WERE FOUND IN THE IMMEDIATE AREA OF WORK ON SAFETY-RELATED PIPING.
(8003 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST (CITE RO REPORT NUMBER, IF APP.)

SYSTEMS AND COMPONENT PROBLEMS:

+ Pressure relief tank overfilled. Shutdown to replace rupture diaphragm.

FACILITY ITEMS (PLANS AND PROCEDURES):

+ Refueling planned for April.

MANAGERIAL ITEMS

None.

PLANT STATUS:

+ Plant operating routinely until refueling in early April.

LAST IE SITE INSPECTION DATE: 03/31-04/04/80+

INSPECTION REPORT NO: 50-206/80-10+

SUMMARY

INSPECTION SUMMARY

+ Inspection on January 29, February 1 & 6-7, 1980
(Report No. 50-206/80-03)
Areas Inspected: Routine, unannounced inspection of plant modifications required as a result of the Three Mile Island Lesson Learned Task Force recommendations (NUREG-0578). The inspection included selective examinations of procedures and representative records, interviews with personnel, observation of work activities, and review of licensee action on unresolved and open items from previous inspections. The inspection involved 37 inspector hours on site by two NRC inspectors.

Results: One item of noncompliance was identified in the area of control of weld filler material.

+ Inspection on January 28 - February 29, 1980
(Report No. 50-206/80-04)
Areas Inspected: Routine, resident inspection of plant operations, surveillance, maintenance, receipt of new fuel, charging pump mini-flow line repair, design changes and modifications, licensee event reports, and onsite review committee activities. The inspection involved 121 inspector hours on site by two NRC inspectors.

Results: Of the eight areas inspected, no items of noncompliance were identified in five areas; one apparent item of noncompliance was identified in the design changes and modifications area (Deficiency - Failure to report reactor protection system setpoints less conservative than those established by the Technical Specifications); two apparent items of noncompliance were identified in the charging pump mini-flow line repair area (Infractions - Use of a nonstandard fitting and failure to dispose of unused welding filler material by bending).

+ Inspection on March 10-13, 1980 (Report No. 50-206/80-05)
Report being prepared; to be reported next month.

+ Inspection on March 3-7, 1980 (Report No. 50-206/80-06)
Areas Inspected: Routine, unannounced inspection of the quality control and confirmatory measurements aspects of the radioactive waste/effluent monitoring program; followup of licensee response to IE Circulars 79-15, 79-21 and Bulletin 80-03; investigation of conditions and licensee action related to LER 80-05 (temperature differential across the circulating cooling water condenser in excess of ETS limits); and independent inspection effort in the areas of spent fuel shipment operations and qualified escort refresher training. The inspection involved a total of 24.5 hours on site by one NRC inspector.

Results: No items of noncompliance or deviations were identified.

+ Inspection on March 18-21, 1980 (Report No. 50-206/80-07)
Areas Inspected: Security Program Audit. The inspection involved 16 inspector hours on site by one NRC inspector.

Results: The licensee was found to be in compliance with NRC requirements and the approved physical security plan.

+ Inspection on January 21-March 28, 1980 (Report No. 50-206/80-08)
Areas Inspected: Physical protection of irradiated fuel shipments. The inspection involved 26 inspector hours on site by four different NRC inspectors.

Results: The licensee was found to be in compliance with NRC requirements.

+ Inspection on March 3-31, 1980 (Report No. 50-206/09)
Report being prepared; to be reported next month.

+ Inspection on March 31-April 4, 1980 (Report No. 50-206/80-10)
Report being prepared; to be reported next month.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

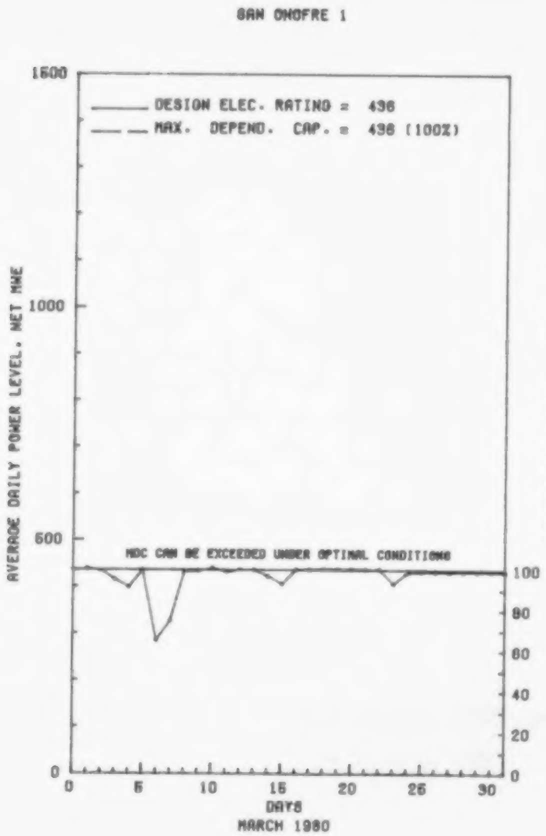
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
80-06/ 01T-0	03-10-80	03-24-80	14 Day LER	During normal operation all three salt water clg pumps failed.

REVIEWED BY OMPA LOEB: P E Bobe **DATE:** 5/12/80

REVIEWED BY NRR DOR: T. V. Wambach **DATE:** 5/13/80

SAN ONOFRE 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-206
2. Utility Contact: R. G. Moran (714) 492-7700
3. Licensed Thermal Power (MWt): 1347
4. Nameplate Rating (Gross MWe): 500 x 0.9 = 450
5. Design Electrical Rating (Net MWe): 436
6. Maximum Dependable Capacity (Gross MWe): 456
7. Maximum Dependable Capacity (Net MWe): 436
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
   NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
   NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	2,184.0	112,160.0
12. Hours Reactor Was Critical	735.2	1,772.6	84,404.8
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	733.2	1,755.3	80,907.7
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	971,712	2,282,577	103,814,051
17. Gross Electrical Energy (MWH)	328,200	764,400	35,435,834
18. Net Electrical Energy (MWH)	314,538	730,743	33,578,109
19. Unit Service Factor	98.5	80.4	72.1
20. Unit Availability Factor	98.5	80.4	72.1
21. Unit Capacity Factor (MDC Net)	97.0	76.7	68.7
22. Unit Capacity Factor (DER Net)	97.0	76.7	68.7
23. Unit Forced Outage Rate	1.5	3.1	9.2

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
   REFUELING, APRIL 11, 1980 - 7 WEEKS
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
51	03/03/80	S	7.3	B	5		HC	HTEXCH	CLEAN CONDENSER WATER BOXES AND TURBINE STOP VALVE TEST
52	03/06/80	F	10.8	A	1	80-011	CJ	VESSEL	OVERFILLED PRT WHILE PURGING, REPLACED PRESSURE RELIEF TANK RUPTURE DIAPHRAGM
53	03/14/80	S	6.0	B	5		HC	HTEXCH	CLEAN CONDENSER WATER BOXES AND TURBINE STOP VALVE TEST.
54	03/23/80	S	5.5	B	5		HF	ZZZZZZ	TURBINE STOP VALVE TEST

SUMMARY

OPERATION WAS AT NEAR FULL POWER DURING THE REPORT PERIOD, WITH LOAD REDUCTIONS FOR CONDENSER WATERBOX CLEANING AND TURBINE STOP VALVE TESTING. ON 03/06/80, THE UNIT WAS TAKEN OFF LINE TO REPLACE THE PRESSURIZER RELIEF TANK RUPTURE DIAPHRAGM.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

ST. LUCIE 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Florida/St. Lucie 12 miles SE of Ft. Pierce
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2560
5.	Design Electrical Rating (MWE Net)	802
6.	Date of Initial Criticality	April 22, 1976
7.	Date of Commercial Operation	December 21, 1976
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Atlantic Ocean

Utility & Contractor Information

10.	Licensee	Florida Power & Light Co.
11.	Corporate Address	9250 W. Flagler Street P. O. Box 529100 Miami, Florida 33152
12.	Corporate Contact	R. E. Uhrig, V.P.
13.	Architect/Engineer	Ebasco
14.	Nuclear Steam Supply System	Combustion Engineering
15.	Constructor	Ebasco

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. D. MARTIN
18.	IE Senior Inspector	E.H. Verdery
19.	IE Principal Inspector	S. A. ELROD
20.	Licensing Project Manager	C. Nelson
21.	Docket Number	50-335
22.	License Number/Date of Issue	DPR-67, March 1, 1976
23.	Public Document Room	Indian River Junior College Library 3209 Virginia Avenue Ft. Pierce, Florida 33450

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

* NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

* UNIT IS IN A REFUELING OUTAGE SINCE MARCH 15, 1980.

* LAST IE SITE INSPECTION DATE: MARCH 24-27, 1980

* INSPECTION REPORT NO: 50-335/80-08

SUMMARY

INSPECTION SUMMARY

* INSPECTION FEBRUARY 19-22 (80-02): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 45 INSPECTOR-HOURS ON SITE IN THE AREAS OF OPERATIONS REVIEW, FACILITY TOURS LICENSEE EVENT REPORT REVIEW AND IE BULLETIN REVIEW. OF THE FOUR AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

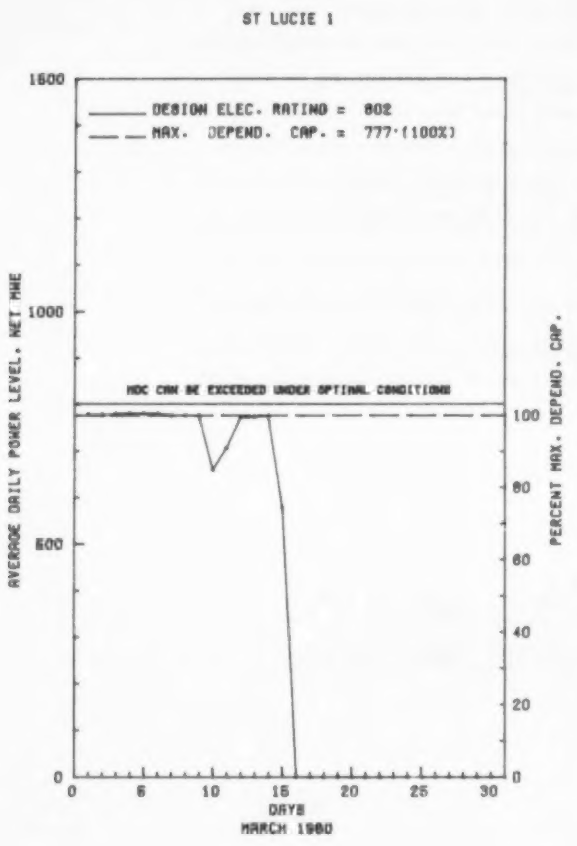
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
80-05/ 03L-0	01/30/80	02/29/80	30 DAY LER	CONTROL ROD DROP DUE TO LOOSE LEAD.
80-06/ 03L-0	02/02/80	03/03/80	30 DAY LER	CONTAINMENT ISOLATION VALVES FOR PRESSURIZER SURGE LINE SAMPLE PIPING COULD NOT BE CLOSED DUE TO CAKED BORIC ACID DEPOSITS.
80-07/ 03L-0	02/04/80	03/03/80	30 DAY LER	CONTROL ROD DROP DUE TO FAILED TIMER MODULE IN COIL POWER PROGRAMMER.
80-08/ 03L-0	02/06/80	03/07/80	30 DAY LER	SAMPLE VALVES FOR SAFETY INJECTION TANKS MISLABELED.

REVIEWED BY OMPA LOEB: MR Beebe **DATE:** 05/12/80

REVIEWED BY NRR DOR: **DATE:** 05/13/80

ST. LUCIE 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-335
2. Utility Contact: V. T. Chilson (305) 552-3824
3. Licensed Thermal Power (MWt): 2560
4. Nameplate Rating (Gross MWe): 1000 x 0.85 = 850
5. Design Electrical Rating (Net MWe): 802
6. Maximum Dependable Capacity (Gross MWe): 822
7. Maximum Dependable Capacity (Net MWe): 777
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	2,184.0	28,728.0
12. Hours Reactor Was Critical	356.7	1,783.1	23,293.0
13. Reactor Reserve Shutdown Hours	.0	.0	129.4
14. Hours Generator On-Line	356.7	1,780.6	22,610.7
15. Unit Reserve Shutdown Hours	.0	.0	32.0
16. Gross Thermal Energy (MWH)	892,672	4,497,283	54,746,849
17. Gross Electrical Energy (MWH)	286,610	1,449,650	17,744,150
18. Net Electrical Energy (MWH)	269,058	1,368,836	16,696,732
19. Unit Service Factor	47.9	81.5	78.7
20. Unit Availability Factor	47.9	81.5	78.8
21. Unit Capacity Factor (MDC Net)	46.5	80.7	74.8
22. Unit Capacity Factor (DER Net)	45.1	78.1	72.5
23. Unit Forced Outage Rate	.0	.9	5.1

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 05/10/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	NONE								

SUMMARY

OPERATION WAS AT NEAR FULL POWER UNTIL 03/15/80, WHEN THE UNIT WAS SHUTDOWN FOR REFUELING, MAINTENANCE, & INSPECTIONS.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

SURRY 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Virginia/Surry 17 miles NW of Newport News
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2441
5.	Design Electrical Rating (MWE Net)	822
6.	Date of Initial Criticality	July 1, 1972
7.	Date of Commercial Operation	December 22, 1972
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	James River

Utility & Contractor Information

10.	Licensee	Virginia Electric and Power Company
11.	Corporate Address	P. O. Box 26666 Richmond, Virginia 23261
12.	Corporate Contact	J. H. Ferguson Exec. V.P. - Power
13.	Architect/Engineer	Stone & Webster
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R.C. Lewis
18.	IE Senior Inspector	P. J. Kellogg
19.	IE Resident Inspector	D. Burke (804) 357-2101
20.	Licensing Project Manager	D. Neighbors
21.	Docket Number	50-280
22.	License Number/Date of Issue	DPR-32, 5/25/72
23.	Public Document Room	Swem Library College of William and Mary Williamsburg, Virginia 23185

INSPECTION STATUS

ENFORCEMENT STATUS

AS REQUIRED BY CRITERION III OF APPENDIX B TO 10 CFR 50 AND SECTION 3 OF THE VEPCO QA MANUAL FOR DESIGN CONTROL, MEASURES AND PROCEDURES WERE ESTABLISHED FOR CONTROLLING THE SUITABILITY OF APPLICATION OF MATERIALS, PARTS, EQUIPMENT, AND PROCESSES THAT ARE ESSENTIAL TO THE SAFETY-RELATED FUNCTIONS OF SYSTEMS AND COMPONENTS, AND FOR ASSURING THAT DESIGN CHANGES, INCLUDING FIELD CHANGES, ARE SUBJECT TO REVIEW AND CONTROL MEASURES COMMENSURATE WITH THOSE APPLIED TO THE ORIGINAL DESIGNS. CONTRARY TO THE ABOVE, WHEN THE UNIT 1 PRESSURIZER POWER-OPERATED RELIEF VALVES (PORV'S) POV-1455C AND -1456 WERE REBUILT ON DECEMBER 28, 1979, THROUGH JANUARY 4, 1980, FLEXITALLIC GASKETS WITH DESIGN RATINGS BELOW THE SPECIFIED REPLACEMENT GASKET RATINGS WERE APPARENTLY INSTALLED IN THE SAFETY-RELATED PORV'S WHEN THE SPECIFIED STOCK WAS DEPLETED. THE SUITABILITY OF THE GASKETS IN THIS APPLICATION WAS NOT REVIEWED AS REQUIRED BY THE APPROPRIATE PLANT MANAGEMENT OR ENGINEERING PERSONNEL.
(8001 INFRACTION)

AS REQUIRED BY TECHNICAL SPECIFICATION 6.4.A.7, DETAILED WRITTEN PROCEDURES WITH APPROPRIATE CHECK-OFF LISTS AND INSTRUCTIONS SHALL BE PROVIDED FOR PREVENTIVE OR CORRECTIVE MAINTENANCE OPERATIONS WHICH WOULD HAVE AN EFFECT ON THE SAFETY OF THE REACTOR. CONTRARY TO THE ABOVE, THE PLANT MAINTENANCE OPERATING PROCEDURES (MOP 5.6 SERIES) FOR FILLING THE REACTOR COOLANT LOOPS WITH WATER DID NOT CONTAIN AN APPROPRIATE CHECK-OFF STEP OR INSTRUCTION TO CLOSE DRAIN VALVE 1-RC-107 FOLLOWING REPLACEMENT OF RC PUMP "A" MOTOR, AND CONSEQUENTLY WAS FOUND OPEN.
(8001 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

RE-ANALYSIS AND REVIEW OF SEISMIC DESIGN DEFICIENCIES ARE CONTINUING.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

UNIT IS CURRENTLY SHUTDOWN FOR MAIN TURBINE INSPECTION AND CORRECTION OF SEISMIC DESIGN ERRORS.

◆ LAST IE SITE INSPECTION DATE: MARCH 13, 1980

◆ INSPECTION REPORT NO: 50-280/80-07

SUMMARY

INSPECTION SUMMARY

◆ INSPECTION DECEMBER 17 - FEBRUARY 4 (80-01): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 180 INSPECTOR-HOURS ON SITE IN THE AREAS OF PLANT OPERATIONS AND OPERATING RECORDS, PLANT MODIFICATION AND MAINTENANCE, FOLLOWUP OF LER'S AND PLANT SECURITY. OF THE FOUR AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE FOUND IN THREE AREAS; TWO APPARENT ITEMS OF NONCOMPLIANCE WERE FOUND IN THE PLANT MAINTENANCE AREA (INFRACTION - INADEQUATE MAINTENANCE PROCEDURES FOR VALVE 1-RC-107 CONTROL, AND INFRACTION - INADEQUATE REVIEW OF SUBSTITUTE GASKETS USED DURING MAINTENANCE OF UNIT 1 PRESSURIZER POWER OPERATED RELIEF VALVES).

INSPECTION FEBRUARY 13-15 (80-03): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED NINETEEN INSPECTOR-HOURS ON SITE IN THE AREA OF SURVEILLANCE TESTING PROGRAM REVIEW. OF THE ONE AREA INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

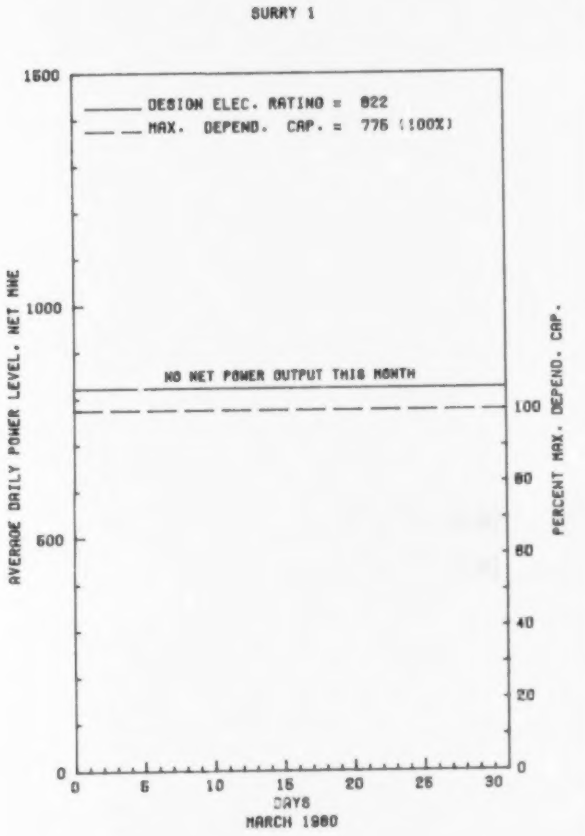
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-043/ 03L-0	12/20/79	01/17/80	30 DAY LER	RADIATION RECORDER RR-175 STOPPED PRINTING FOR ABOUT 13 MINUTES DUE TO A CONTROL STRING ON DRIVE WHEEL.
79-044/ 03L-0	12/30/79	01/17/80	30 DAY LER	AN OPERABILITY CHECK OF #3 EMERGENCY DIESEL GENERATOR DETECTED A NOISE IN THE TURBO-CHARGER AREA.
80-04/ 01T-0	01/08/80	01/21/80	14 DAY LER	AUXILIARY FEEDWATER PUMP DISCHARGE CHECK VALVE 1-FW-89 STUCK OPEN.
80-05/ 03L-0	01/20/80	02/07/80	30 DAY LER	ONE HEAT TRACE CIRCUIT TO THE BORIC ACID FILTER INOPERABLE.
80-06/ 03X-1	01/15/80	02/07/80	30 DAY SUPPLEMENT	LIQUID WASTE TANK 11A BEING PUMPED TO TANK 11B DURING RELEASE FROM TANK 11B.
80-07/ 03L-0	01/16/80	02/07/80	30 DAY LER	ONE HEAT TRACE CIRCUIT TO THE BORIC ACID SUPPLY INOPERABLE.
80-08/ 01T-0	01/16/80	01/30/80	14 DAY LER	ONE BORATION FLOW PATH OUT OF SERVICE BECAUSE OF CLOSED VALVE 1-CH-83.
80-09/ 03L-0	01/23/80	02/19/80	30 DAY LER	BOTH BORIC ACID TRANSFER PUMPS INOPERABLE RESULTING IN UNIT RAMPDOWN UNTIL ONE PUMP REPAIRED.
80-10/ 01T-0	01/31/80	02/14/80	14 DAY LER	REANALYSIS OF COMPONENT COOLING PIPING TO A RESIDUAL HEAT REMOVAL HEAT EXCHANGER INDICATES OVERSTRESS DURING SEISMIC EVENT.
80-11/ 03L-0	01/28/80	02/26/80	30 DAY LER	ONE HEAT TRACE CIRCUIT FOR BORIC ACID TRANSFER PUMP 2A INOPERABLE.
80-12/ 03L-0	02/02/80	02/28/80	30 DAY LER	HEAT TRACE INOPERABLE TO BORON INJECTION TANK RECIRC LINE RESULTED IN UNIT RAMPDOWN SINCE REPAIRS TOOK LONGER THAN 2 HOURS.
80-13/ 03L-0	02/06/80	03/03/80	30 DAY LER	ACCUMULATOR TANK SI-TK-1A BELOW LEVEL AND PRESSURE LIMITS DUE TO MALFUNCTIONING VALVE.
80-018/ 01T-0	02/19/80	02/28/80	14 DAY LER	PIPING PENETRATIONS FOR TWO PIPES INSIDE CONTAINMENT OVER-STRESSED.

REVIEWED BY OMPA LOEB: ________ **DATE:** 5/12/80

REVIEWED BY NRR DOR: Don Neighbors **DATE:** 5/12/80

SURRY 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-280
2. Utility Contact: O. J. Costello (804) 357-3184
3. Licensed Thermal Power (MWt): 2441
4. Nameplate Rating (Gross MWe): 942 x 0.9 = 848
5. Design Electrical Rating (Net MWe): 822
6. Maximum Dependable Capacity (Gross MWe): 811
7. Maximum Dependable Capacity (Net MWe): 775
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	2,184.0	63,768.0
12. Hours Reactor Was Critical	.0	1,016.2	39,767.8
13. Reactor Reserve Shutdown Hours	.0	182.6	3,731.5
14. Hours Generator On-Line	.0	1,010.4	38,915.4
15. Unit Reserve Shutdown Hours	.0	183.6	3,736.2
16. Gross Thermal Energy (MWH)	0	2,438,340	90,153,212
17. Gross Electrical Energy (MWH)	0	785,655	29,466,153
18. Net Electrical Energy (MWH)	0	746,810	27,973,709
19. Unit Service Factor	.0	46.3	61.0
20. Unit Availability Factor	.0	54.7	66.9
21. Unit Capacity Factor (MDC Net)	.0	44.1	56.6
22. Unit Capacity Factor (DER Net)	.0	41.6	53.4
23. Unit Forced Outage Rate	100.0	53.7	26.0

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUELING - 09/80 - 6 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: 04/28/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
8-4	02/19/80	F	744.0	D	4	80-018			CONTINUATION OF SHUTDOWN FOR PIPING RESTRAINT MODIFICATION AND TURBINE ROTOR INSPECTION WHICH BEGAN 2/19/80.

SUMMARY

THE UNIT REMAINED SHUTDOWN THROUGHOUT THE REPORT PERIOD FOR PIPING MODIFICATIONS AND TURBINE INSPECTION.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

SURRY 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Virginia/Surry 17 miles NW of Newport News
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2441
5.	Design Electrical Rating (MWE Net)	822
6.	Date of Initial Criticality	March 7, 1973
7.	Date of Commercial Operation	May 1, 1973
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	James River

Utility & Contractor Information

10.	Licensee	Virginia Electric and Power Company
11.	Corporate Address	P. O. Box 26666 Richmond, Virginia 23261
12.	Corporate Contact	J. H. Ferguson Exec. V.P. - Power
13.	Architect/Engineer	Stone & Webster
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R.C. Lewis
18.	IE Senior Inspector	P. J. Kellog
19.	IE Resident Inspector	D. Burke (804) 357-2101
20.	Licensing Project Manager	D. Neighbors
21.	Docket Number	50-281
22.	License Number/Date of Issue	DPR-37, 1/29/73
23.	Public Document Room	Swem Library College of William and Mary Williamsburg, Virginia 23185

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

SYSTEM MODIFICATIONS FOR SEISMIC SUPPORT REANALYSIS AND NUREG 0578 ARE CONTINUING.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

UNIT IS CURRENTLY SHUT DOWN AND DEFUELED. THE STEAM GENERATOR REPLACEMENT PROJECT COMPLETED.

♦ LAST IE SITE INSPECTION DATE: MARCH 13, 1980

♦ INSPECTION REPORT NO: 50-281/80-07

SUMMARY

INSPECTION SUMMARY

♦ INSPECTION FEBRUARY 19-22 AND 26-28 (80-01): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 90 INSPECTOR-HOURS ON SITE IN THE AREAS OF CONTAINMENT INTEGRATED LEAKAGE RATE TESTING. OF THE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION DECEMBER 17 - FEBRUARY 4 (80-03): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 180 INSPECTOR-HOURS ON SITE IN THE AREAS OF PLANT OPERATIONS AND OPERATING RECORDS, PLANT MODIFICATION AND MAINTENANCE, FOLLOWUP OF LER'S AND PLANT SECURITY. OF THE FOUR AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE FOUND IN THREE AREAS; TWO APPARENT ITEMS OF NONCOMPLIANCE WERE FOUND IN THE PLANT MAINTENANCE AREA (INFRACTION - INADEQUATE MAINTENANCE PROCEDURES FOR VALVE 1-RC-107 CONTROL, AND INFRACTION - INADEQUATE REVIEW OF SUBSTITUTE GASKETS USED DURING MAINTENANCE OF UNIT 1 PRESSURIZER POWER OPERATED RELIEF VALVES).

INSPECTION FEBRUARY 13-15 (80-05): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED NINETEEN INSPECTOR-HOURS ON SITE IN THE AREA OF SURVEILLANCE TESTING PROGRAM REVIEW. OF THE ONE AREA INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION MARCH 9-13 (80-07): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 89 INSPECTOR-HOURS ON SITE IN THE AREA OF WITNESSING THE INTEGRATED LEAK RATE TESTING. IN THE AREA INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT

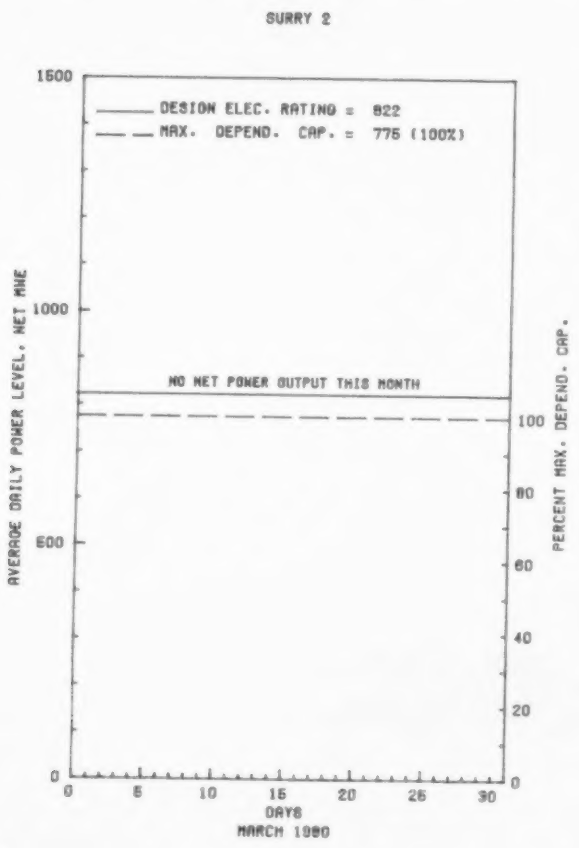
NONE.

REVIEWED BY OMPA LOEB: R S Bobe **DATE:** 5/14/80

REVIEWED BY NRR DOR: Don Neighbors **DATE:** 5/12/80

SURRY 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-281
2. Utility Contact: O. J. Costello (804) 357-3184
3. Licensed Thermal Power (MWt): 2441
4. Nameplate Rating (Gross MWe): 942 x 0.9 = 848
5. Design Electrical Rating (Net MWe): 822
6. Maximum Dependable Capacity (Gross MWe): 811
7. Maximum Dependable Capacity (Net MWe): 775
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	2,184.0	60,648.0
12. Hours Reactor Was Critical	.0	.0	34,499.9
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	.0	.0	33,996.2
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	0	79,196,083
17. Gross Electrical Energy (MWH)	0	0	25,868,844
18. Net Electrical Energy (MWH)	0	0	24,536,605
19. Unit Service Factor	.0	.0	56.1
20. Unit Availability Factor	.0	.0	56.1
21. Unit Capacity Factor (MDC Net)	.0	.0	52.2
22. Unit Capacity Factor (DER Net)	.0	.0	49.2
23. Unit Forced Outage Rate	.0	.0	21.0

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 05/22/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-2	02/04/79	S	744.0	D	4				MODIFICATIONS BEING MADE AS A RESULT OF SHOW CAUSE ORDER FOR SEISMIC ANALYSIS AND IE BULLETIN 79-14.

SUMMARY

THE UNIT REMAINED SHUTDOWN THROUGHOUT THE REPORT PERIOD FROM STEAM GENERATOR REPLACEMENT OUTAGE. MODIFICATIONS BEING MADE AS A RESULT OF SHOW CAUSE ORDER FOR SEISMIC ANALYSIS AND IE BULLETIN 79-14.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

THREE MILE ISLAND 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Pennsylvania/Dauphin 10 miles SE of Harrisburg
2.	Electric Reliability Council	Mid-Atlantic Area Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2535
5.	Design Electrical Rating (MWE Net)	819
6.	Date of Initial Criticality	June 5, 1974
7.	Date of Commercial Operation	September 2, 1974
8.	Condenser Cooling Method	Closed
9.	Condenser Cooling Water Source	Susquehanna River

Utility & Contractor Information

10.	Licensee	Metropolitan Edison Company
11.	Corporate Address	P. O. Box 542 Reading, Pennsylvania
12.	Corporate Contact	R. C. Arnold, Sr. Vice-President 260 Cherry Hill Rd. Parsippany, NJ 07054
13.	Architect/Engineer	Gilbert Associates, Inc.
14.	Nuclear Steam Supply System	B&W
15.	Constructor	UE&C

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Senior Inspector	R. R. Keimig
19.	NRC TMI Support Staff	D. Haverkamp (717) 782-3950
20.	Licensing Project Manager	D. C. DiIanni
21.	Docket Number	50-289
22.	License Number/Date of Issue	DPR-50, 4/19/74
23.	Public Document Room	Government Publications Section State Library of Pennsylvania Education Building Commonwealth and Walnut Street Harrisburg, Pennsylvania 17126

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

None

FACILITY ITEMS (PLANS AND PROCEDURES):

None

MANAGERIAL ITEMS:

Mr. R. M. Toole was appointed to the new position of TMI-1 Manager effective February 1, 1980.

PLANT STATUS:

The plant remains shutdown by NRC order, pending completion of modifications and other actions related to the TMI-2 accident.

LAST IE SITE INSPECTION DATE: 2/20/80

INSPECTION REPORT NO: 50-289/80-02

SUMMARY

INSPECTION SUMMARY

No inspection reports issued.

REPORTS RECEIVED FROM LICENSEE

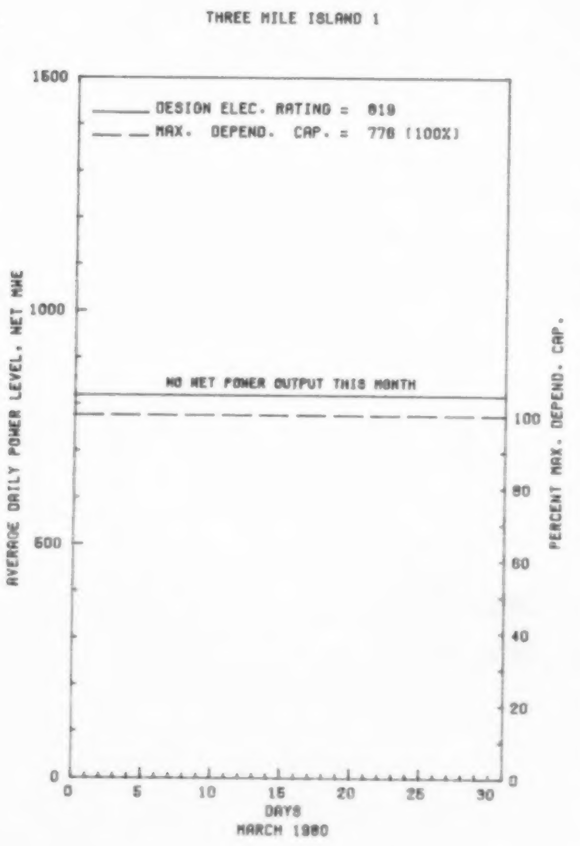
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-017/01T	11/08/79	12/11/79	14 Day LER	Inadequate bolt installation and design in rigid support anchor for decay heat removal cooling piping

REVIEWED BY OMPA LOEB: P E Bobe **DATE:** 5/12/80

REVIEWED BY NRR DOR: D C DiIanni **DATE:** 5/13/80

THREE MILE ISLAND 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-289
2. Utility Contact: D. G. Mitchell (215) 921-6579
3. Licensed Thermal Power (MWt): 2535
4. Nameplate Rating (Gross MWe): 968 x 0.9 = 871
5. Design Electrical Rating (Net MWe): 819
6. Maximum Dependable Capacity (Gross MWe): 840
7. Maximum Dependable Capacity (Net MWe): 776
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
   NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
    NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	2,184.0	48,913.0
12. Hours Reactor Was Critical	.0	.0	31,731.8
13. Reactor Reserve Shutdown Hours	.0	.0	838.5
14. Hours Generator On-Line	.0	.0	31,180.9
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	0	76,531,071
17. Gross Electrical Energy (MWH)	0	0	25,484,330
18. Net Electrical Energy (MWH)	0	0	23,840,053
19. Unit Service Factor	.0	.0	63.7
20. Unit Availability Factor	.0	.0	63.7
21. Unit Capacity Factor (MDC Net)	.0	.0	62.8
22. Unit Capacity Factor (DER Net)	.0	.0	59.5
23. Unit Forced Outage Rate	100.0	100.0	24.9

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
    NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
1	02/17/79	F	744.0	D	4				REGULATORY RESTRAINT ORDER

SUMMARY

THE UNIT REMAINS SHUTDOWN DUE TO REGULATORY CONSIDERATION OF THE TMI-2 ACCIDENT.

LEGEND

[1] Type:
F–Forced
S–Scheduled

[2] Reason:
A–Equipment Failure
B–Maintenance or Test
C–Refueling
D–Regulatory Restriction
E–Operator Training & License Examination
F–Administrative
G–Operational Error
H–Other

[3] Method:
1–Manual
2–Manual Scram
3–Automatic Scram
4–Continuations
5–Load Reduction
9–Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

THREE MILE ISLAND 2

NRC POWER RESTRICTIONS: SEE "LICENSE STATUS"

FACILITY DATA

Facility Description		
1.	Location (state/county) (nearest population center)	Pennsylvania/Dauphin 10 miles SE of Harrisburg
2.	Electric Reliability Council	Mid-Atlantic Area Council
3.	Type of Reactor	PWR
4.	Licensed Power Level (MWT)	2772
5.	Design Electrical Rating (MWE Net)	906
6.	Date of Initial Criticality	3/28/78
7.	Date of Commercial Operation	12/30/78
8.	Condenser Cooling Method	CLOSED
9.	Condenser Cooling Water Source	Susquehanna River
Utility & Contractor Information		
10.	Licensee	Metropolitan Edison Company
11.	Corporate Address	P. O. Box 542 Reading, PA 19603
12.	Corporate Contact	R. C. Arnold, Sr. Vice-President 260 Cherry Hill Rd. Parsippany, NJ 07054
13.	Architect/Engineer	Burns & Roe
14.	Nuclear Steam Supply System	B&W
15.	Constructor	UE&C
Regulatory Information		
16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Senior Inspector	R. R. Keimig
19.	NRC-TMI Support Staff	R. Conte (717) 782-3950
20.	TMI Program Director	B. Snyder
21.	Docket Number	50-320
22.	License Number/Date of Issue	DPR-73/2; 2/8/78
23.	Public Document Room	Government Publications Section State Library of Pennsylvania Education Building Commonwealth & Walnut Streets Harrisburg, Pennsylvania 17126

T-4

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

```
INSPECTION STATUS

OTHER ITEMS OF INTEREST
SYSTEMS AND COMPONENT PROBLEMS:
Recovery operations from the 3/28/79 incident.
FACILITY ITEMS (PLANS AND PROCEDURES):
None
MANAGERIAL ITEMS:
None
PLANT STATUS:
No significant information to report.
LAST IE SITE INSPECTION DATE:  1/31/80
     INSPECTION REPORT NO:  50-320/80-01
SUMMARY
INSPECTION SUMMARY
No inspections completed.
```

LICENSE STATUS

By Order for Modification of License dated July 20, 1979, the licensee's authority to operate the facility was suspended and the licensee's authority was limited to maintenance of the facility in the present shutdown cooling mode (44 F.R. 45271).

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
	NONE			

REVIEWED BY OMPA LOEB: R Bote **DATE:** 5/12/80

REVIEWED BY NRR DOR: D. L. Brinkman **DATE:** 5/13/80

THREE MILE ISLAND 2

AVERAGE DAILY POWER LEVEL (MWe)

OPERATING STATUS

1. Docket Number: 50-320
2. Utility Data Prepared By: R. A. Lengel (215) 921-6581
3. Licensed Thermal Power (MWt): 2772
4. Nameplate Rating (Gross MWe): 1068 X 0.9 = 961
5. Design Electrical Rating (Net MWe): 906
6. Maximum Dependable Capacity (Gross MWe): 0
7. Maximum Dependable Capacity (Net MWe): 0
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:
None
9. Power Level To Which Restricted, If Any (Net MWe): None
10. Reasons For Restrictions, If Any:

This Month Yr

11. Hours In Reporting Period
12. Number Of Hours Reactor Was Critical
13. Reactor Reserve Shutdown Hours
14. Hours Generator On-Line
15. Unit Reserve Sh
16. Gro

(MWH)

19

20. U Availability Factor
21. Unit Capacity Factor (Using MDC Net) 1/
22. Unit Capacity Factor (Using DER Net) 1/
23. Unit Forced Outage Rate
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
None
25. If Shut Down At End Of Report Period, Estimated Date of Startup: N/A

NO DECISION ON FUTURE OPERATION
SHUTDOWN INDEFINITELY

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
SEE SUMMARY BELOW.									

SUMMARY

THE UNIT HAS REMAINED SHUTDOWN SINCE THE 03/28/79 ACCIDENT, WHICH CAUSED EXTENSIVE CORE DAMAGE.

AUTHORITY TO OPERATE FACILITY SUSPENDED BY NRC ORDER DATED JULY 20, 1979,
WHICH REQUIRES LICENSEE TO MAINTAIN FACILITY IN A SHUTDOWN CONDITION.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

TROJAN

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Oregon/Columbia 42 miles N of Portland
2.	Electric Reliability Council	Western Systems Coordinating Council
3.	Type of Reactor	PWR
4.	Licensed Power Level (MWT)	3411
5.	Design Electrical Rating (MWE Net)	1130
6.	Date of Initial Criticality	December 15, 1975
7.	Date of Commercial Operation	May 20, 1976
8.	Condenser Cooling Method	Natural draft cooling tower
9.	Condenser Cooling Water Source	Columbia River

Utility & Contractor Information

10.	Licensee	Portland General Electric Co.
11.	Corporate Address	121 S. W. Salmon St. Portland, Oregon 97204
12.	Corporate Contact	C. Goodwin, Jr., Asst. V.P.
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Multiple

Regulatory Information

16.	IE Region responsible	V
17.	IE Branch Chief	J. L. Crews
18.	IE Senior Inspector	D. M. Sternberg
19.	IE Resident Inspector	M. H. Malmros (503) 556-3751
20.	Licensing Project Manager	C. M. Trammell
21.	Docket Number	50-344
22.	License Number/Date of Issue	NPF-1, 11/21/75
23.	Public Document Room	Columbia County Courthouse Law Library Circuit Court Room St. Helens, Oregon 97501

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST (CITE RO REPORT NUMBER, IF APP.)

SYSTEMS AND COMPONENT PROBLEMS:

None.

FACILITY ITEMS (PLANS AND PROCEDURES):

1. The phase two Public Hearing on control building modifications is planned for April 1980.
2. A two and one-half month refueling outage is scheduled to commence in mid-April 1980.

MANAGERIAL ITEMS:

None.

PLANT STATUS

* The plant operated at power throughout the month of March.

LAST IE SITE INSPECTION DATE: 03/03-31/80*

INSPECTION REPORT NO: 50-344/80-06*

SUMMARY

INSPECTION SUMMARY:

* Inspection on February 15, 1980 (Report No. 50-344/80-03) Areas Inspected: Headquarters and region review of the licensee's approved security plan. The inspection involved six hours by two NRC inspectors.

Results: The licensee was found to be in compliance with NRC requirements.

* Inspection on February 1-29, 1980 (Report No. 50-344/80-04) Areas Inspected: Routine inspections of plant operations, maintenance, physical security, training, refueling preparations, and procedures. The inspection involved 137 inspector hours by the NRC Resident Inspectors.

Results: No items of noncompliance or deviations were identified.

* Inspection on March 17-21, 1980 (Report No. 50-344/80-05) Report being prepared; to be reported next month.

* Inspection on March 3-31, 1980 (Report No. 50-344/80-06) Report being prepared; to be reported next month.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

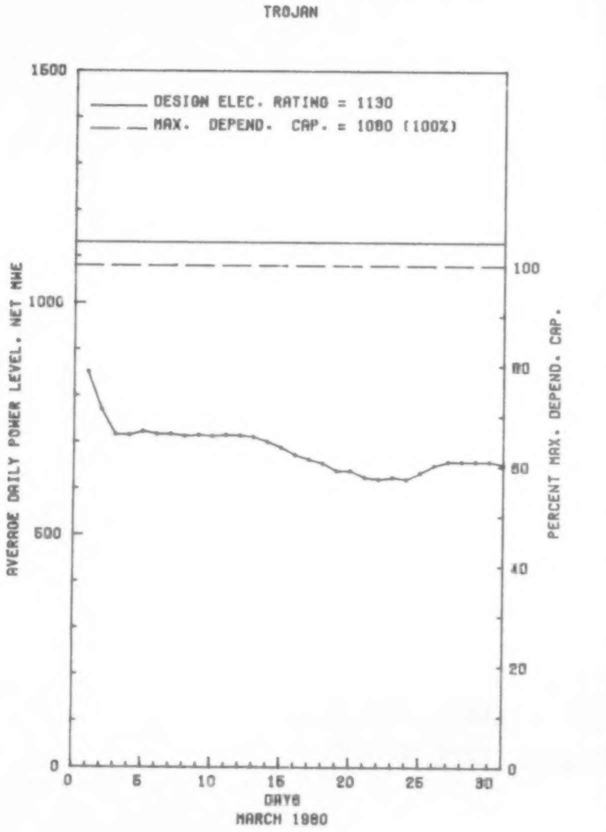
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
80-01/ 03L-0	02-04-80	02-29-80	30 Day LER	B DBA sequencer failed to start SI and RHR pumps.
80-03/ 03L-0	02-19-80	03-10-80	30 Day LER	Pressurizer pressure transmitter setpoint drift.

REVIEWED BY OMPA LOEB: PE Bote DATE: 5/14/80

REVIEWED BY NRR DOR: CM Trammell DATE: 5/12/80

TROJAN

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-344
2. Utility Contact: G. G. Bair (503) 556-3713 x234
3. Licensed Thermal Power (MWt): 3411
4. Nameplate Rating (Gross MWe): 1280 x 0.95 = 1216
5. Design Electrical Rating (Net MWe): 1130
6. Maximum Dependable Capacity (Gross MWe): 1122
7. Maximum Dependable Capacity (Net MWe): 1080
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	2,184.0	31,392.0
12. Hours Reactor Was Critical	744.0	2,164.3	18,462.3
13. Reactor Reserve Shutdown Hours	.0	.0	2,171.8
14. Hours Generator On-Line	744.0	2,159.4	17,787.2
15. Unit Reserve Shutdown Hours	.0	.0	1,508.7
16. Gross Thermal Energy (MWH)	1,713,743	6,270,315	55,165,925
17. Gross Electrical Energy (MWH)	541,030	2,025,625	18,011,040
18. Net Electrical Energy (MWH)	508,723	1,930,261	16,963,580
19. Unit Service Factor	100.0	98.9	56.7
20. Unit Availability Factor	100.0	98.9	61.5
21. Unit Capacity Factor (MDC Net)	63.3	81.8	50.0
22. Unit Capacity Factor (DER Net)	60.5	78.2	47.8
23. Unit Forced Outage Rate	.0	1.1	28.9

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
REFUELING, APRIL 9, 1980; 78 DAYS
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	NONE								

SUMMARY

OPERATION WAS ROUTINE IN AN END-OF-CYCLE COASTDOWN, WITH NO OUTAGES OR FURTHER REDUCTIONS REPORTED.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

best copy available

TURKEY POINT 3

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Florida/Dade 25 miles S of Miami
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2200
5.	Design Electrical Rating (MWE Net)	693
6.	Date of Initial Criticality	October 20, 1972
7.	Date of Commercial Operation	December 14, 1972
8.	Condenser Cooling Method	Recirculating Canal System
9.	Condenser Cooling Water Source	Biscayne Bay

Utility & Contractor Information

10.	Licensee	Florida Power and Light Company
11.	Corporate Address	9250 West Flagler Street P. O. Box 013100 Miami, Florida 33101
12.	Corporate Contact	R. E. Uhrig, V.P. Nuclear Affairs
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. Lewis
18.	IE Senior Inspector	E. H. Verdery
19.	IE Resident Inspector	R. J. Vogt-Lowell (305) 245-7669
20.	Licensing Project Manager	M. Grotenhuis (301) 492-7128
21.	Docket Number	50-250
22.	License Number/Date of Issue	DPR-31, 7/19/72
23.	Public Document Room	Environmental and Urban Affairs Library Florida International University Miami, Florida 33199

INSPECTION STATUS

ENFORCEMENT STATUS

NRC LETTER OF FEBRUARY 5, 1978, "FIRE PROTECTION GUIDELINES", ATTACHMENT 6; FLORIDA POWER AND LIGHT COMPANY 'S (FP&L) RESPONSE LETTER L-78-197. FIRE PROTECTION GUIDELINES, ENCLOSURE 1, ATTACHMENT 6 AND NRC CONFIRMING LETTER OF JUNE 30, 1978, "FIRE PROTECTION GUIDELINES" COMPRISE A COMMITMENT MADE AND ACCEPTED THAT FP&L ESTABLISH A FIRE PROTECTION QUALITY ASSURANCE (FPQA) PROGRAM BASED ON DRAFT REGULATORY GUIDE 1.120, JUNE, 1976 WITH A "LEVELS OF QUALITY" APPROACH. DURING THE WEEK OF JANUARY 7-11, 1980, THE LICENSEE COULD FURNISH NO EVIDENCE OF COMPLETED FPQA PROGRAM DEVELOPMENT OR IMPLEMENTATION.
(8003 DEVIATION)

AS REQUIRED BY 10 CFR 50, APPENDIX B, CRITERION XVII, AND TOPICAL QUALITY REQUIREMENT 17.0 OF THE TOPICAL QUALITY ASSURANCE REPORT NO. FPLTQAR 1-76, AS IMPLEMENTED IN PART BY PARAGRAPH 8.4.5 OF ADMINISTRATIVE PROCEDURE A.P. 190.14, INCORRECT ENTRIES ON QA RECORDS SHALL BE CORRECTED BY DRAWING A SINGLE LINE THROUGH THE INCORRECT ENTRY AND THEN ADDING THE CORRECT ITEM. THE PERSON MAKING THE CORRECTION SHALL INITIAL AND DATE THE CORRECTION. CONTRARY TO THE ABOVE, CORRECTIONS ON UNIT 3 CLEARANCE ORDERS 12-57 AND 1-261 ISSUED 12-1-79 AND 1-29-80 RESPECTIVELY WERE MADE WITHOUT THE REQUIRED INITIALS AND/OR DATES BEING INDICATED.
(8004 DEFICIENCY)

AS REQUIRED BY SECTION 6.8.1 OF THE TECHNICAL SPECIFICATIONS, WRITTEN PROCEDURES AND ADMINISTRATIVE POLICIES THAT MEET OR EXCEED THE REQUIREMENTS AND RECOMMENDATIONS OF SECTION 5.1 AND 5.3 OF ANSI N18.7-1972 AND APPENDIX A OF USNRC REGULATORY GUIDE 1.33 SHALL BE IMPLEMENTED. CONTRARY TO THE ABOVE, THE FOLLOWING REPRESENTS FAILURE TO IMPLEMENT EXISTING PROCEDURES: EQUIPMENT CLEARANCE ORDER FORMS 1-259, 1-225, 1-192, 1-267 AND 1-268 ISSUED DURING JANUARY 1980 WERE NOT FILLED OUT IN THE MANNER REQUIRED BY A.P. 0103.4, "IN-PLANT EQUIPMENT CLEARANCE ORDERS."
(8004 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

NORMAL OPERATION AT 100% POWER.

◆ LAST IE SITE INSPECTION DATE: MARCH 26-29, 1980

◆ INSPECTION REPORT NO: 50-250/80-08

SUMMARY

INSPECTION SUMMARY

◆ INSPECTION JANUARY 7-31 (80-04): THIS ROUTINE INSPECTION INVOLVED 60 INSPECTOR-HOURS ON SITE IN THE AREAS OF PLANT OPERATIONS; MAINTENANCE ACTIVITIES; REACTOR COOLANT PUMP SEAL REPLACEMENT; EQUIPMENT CLEARANCE ORDERS; QUALITY ASSURANCE RECORDS; LICENSEE ACTION ON IE BULLETINS. OF THE SIX AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN FOUR AREAS; TWO APPARENT ITEMS OF NONCOMPLIANCE WERE FOUND IN TWO AREAS (INFRACTION - FAILURE TO FOLLOW PROCEDURES; DEFICIENCY - FAILURE TO FOLLOW TOPICAL QUALITY REQUIREMENT 17.0 AS IMPLEMENTED BY AP 190.14).

INSPECTION FEBRUARY 4-7 (80-06): THIS SPECIAL, ANNOUNCED INSPECTION INVOLVED 34 INSPECTOR-HOURS ON SITE IN THE AREAS OF SMALL BREAK LOSS OF COOLANT PROCEDURES AND TRAINING. WITHIN THE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION FEBRUARY 19-22 (80-07): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 12 INSPECTOR-HOURS ON SITE IN THE AREAS OF STEAM GENERATOR REPAIR FOR TURKEY POINT STATION. OF THE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

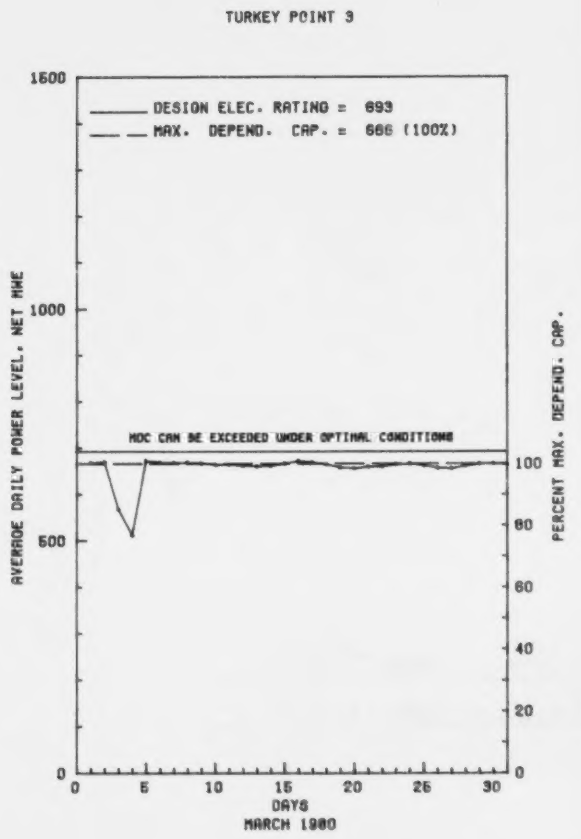
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
80-02/03L-0	02/03/80	03/03/80	30 DAY LER	UNIT 3 AND UNIT 4 CST VOLUMES LESS THAN 185000 GAL REQUIRED BY TS 3.8.1.C.

REVIEWED BY OMPA LOEB: PE Bole DATE: 5/12/80

REVIEWED BY NRR DOR: MGrotenhuis DATE: 5/12/80

TURKEY POINT 3

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-250
2. Utility Contact: V. T. Chilson (305) 552-3824
3. Licensed Thermal Power (MWt): 2200
4. Nameplate Rating (Gross MWe): 894 x 0.85 = 760
5. Design Electrical Rating (Net MWe): 693
6. Maximum Dependable Capacity (Gross MWe): 700
7. Maximum Dependable Capacity (Net MWe): 666
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

ITEM 21 CALCULATED BY USING A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	2,184.0	64,185.6
12. Hours Reactor Was Critical	742.9	1,379.6	48,649.0
13. Reactor Reserve Shutdown Hours	.0	.0	213.3
14. Hours Generator On-Line	738.2	1,250.4	46,940.6
15. Unit Reserve Shutdown Hours	.0	.0	121.8
16. Gross Thermal Energy (MWH)	1,612,394	2,645,998	94,114,632
17. Gross Electrical Energy (MWH)	513,485	848,545	29,929,565
18. Net Electrical Energy (MWH)	488,351	800,363	28,322,457
19. Unit Service Factor	99.2	57.3	73.1
20. Unit Availability Factor	99.2	57.3	73.3
*21. Unit Capacity Factor (MDC Net)	98.6	55.0	66.8
22. Unit Capacity Factor (DER Net)	94.7	52.9	63.7
23. Unit Forced Outage Rate	.8	2.5	2.7

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
STEAM GEN. TUBE INSPEC. PROGRAM - SEPT. 15 - OCT. 13, 1980
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
8	03/03/80	F	5.8	B	4		CB	MOTORX	UNIT WAS REMOVED FROM SERVICE DUE TO LOW OIL LEVEL IN REACTOR COOLANT PUMP MOTOR NO. 3C. CORRECTIVE ACTIONS INCLUDED ADDING OIL.

SUMMARY

OPERATION WAS AT NEAR FULL POWER DURING THE REPORT PERIOD, EXCEPT FOR AN OUTAGE ON 03/03/80 TO ADD OIL TO THE NO. 3C REACTOR COOLANT PUMP MOTOR.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

TURKEY POINT 4

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Florida/Dade 25 miles S of Miami
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2200
5.	Design Electrical Rating (MWE Net)	693
6.	Date of Initial Criticality	June 11, 1973
7.	Date of Commercial Operation	September 7, 1973
8.	Condenser Cooling Method	Recirculating Canal System
9.	Condenser Cooling Water Source	Biscayne Bay

Utility & Contractor Information

10.	Licensee	Florida Power and Light Company
11.	Corporate Address	9250 West Flagler Street P. O. Box 013100 Miami, Florida 33101
12.	Corporate Contact	R. E. Uhrig, V. P. Nuclear Affairs
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. Lewis
18.	IE Senior Inspector	E.H. Verdery
19.	IE Resident Inspector	R. J. Vogt-Lowell (305) 245-7669
20.	Licensing Project Manager	M. Grotenhuis (301) 492-7128
21.	Docket Number	50-251
22.	License Number/Date of Issue	DPR-41, 4/10/73
23.	Public Document Room	Environmental and Urban Affairs Library Florida International University Miami, Florida 33199

INSPECTION STATUS

ENFORCEMENT STATUS

NRC LETTER OF FEBRUARY 5, 1978, "FIRE PROTECTION GUIDELINES", ATTACHMENT 6; FLORIDA POWER AND LIGHT COMPANY 'S (FP&L) RESPONSE LETTER L-78-197, FIRE PROTECTION GUIDELINES, ENCLOSURE 1, ATTACHMENT 6 AND NRC CONFIRMING LETTER OF JUNE 30, 1978, "FIRE PROTECTION GUIDELINES" COMPRISE A COMMITMENT MADE AND ACCEPTED THAT FP&L ESTABLISH A FIRE PROTECTION QUALITY ASSURANCE (FPQA) PROGRAM BASED ON DRAFT REGULATORY GUIDE 1.120, JUNE, 1976 WITH A "LEVELS OF QUALITY" APPROACH. DURING THE WEEK OF JANUARY 7-11, 1980, THE LICENSEE COULD FURNISH NO EVIDENCE OF COMPLETED FPQA PROGRAM DEVELOPMENT OR IMPLEMENTATION. (8003 DEVIATION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

UNIT DUE TO SHUT DOWN FOR STEAM GENERATOR TUBE INSPECTION.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

UNIT OPERATING AT 100% POWER.

♦ LAST IE SITE INSPECTION DATE: MARCH 26-29, 1980

♦ INSPECTION REPORT NO: 50-251/80-08

SUMMARY

INSPECTION SUMMARY

♦ INSPECTION JANUARY 7-31 (80-04): THIS ROUTINE INSPECTION INVOLVED 60 INSPECTOR-HOURS ON SITE IN THE AREAS OF PLANT OPERATIONS; MAINTENANCE ACTIVITIES; EQUIPMENT CLEARANCE ORDERS; QUALITY ASSURANCE RECORDS; LICENSEE ACTION ON IE BULLETINS. OF THE FIVE AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION FEBRUARY 4-7 (80-06): THIS SPECIAL, ANNOUNCED INSPECTION INVOLVED 35 INSPECTOR-HOURS ON SITE IN THE AREAS OF SMALL BREAK LOSS OF COOLANT PROCEDURES AND TRAINING. WITHIN THE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION FEBRUARY 19-22 (80-07): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 12 INSPECTOR-HOURS ON SITE IN THE AREAS OF STEAM GENERATOR REPAIR FOR TURKEY POINT STATION. OF THE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERF IDENTIFIED.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

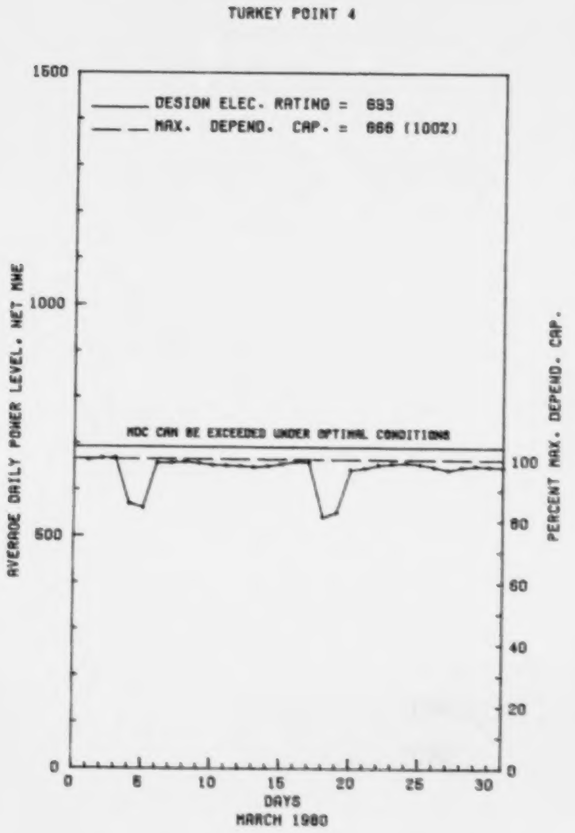
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
		NONE		

REVIEWED BY OMPA LOEB: RE Bobe **DATE:** 5/12/80

REVIEWED BY NRR DOR: M Grotenhuis **DATE:** 5/12/80

TURKEY POINT 4

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-251
2. Utility Contact: V. T. Chilson (305) 552-3824
3. Licensed Thermal Power (MWt): 2200
4. Nameplate Rating (Gross MWe): 894 x 0.85 = 760
5. Design Electrical Rating (Net MWe): 693
6. Maximum Dependable Capacity (Gross MWe): 700
7. Maximum Dependable Capacity (Net MWe): 666
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

ITEM 21 CALCULATED BY USING A WEIGHTED AVERAGE.

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	2,184.0	57,913.0
12. Hours Reactor Was Critical	743.5	2,043.0	42,870.8
13. Reactor Reserve Shutdown Hours	.0	12.3	166.6
14. Hours Generator On-Line	739.8	1,995.9	41,178.4
15. Unit Reserve Shutdown Hours	.0	12.3	31.2
16. Gross Thermal Energy (MWH)	1,604,686	4,310,750	85,529,730
17. Gross Electrical Energy (MWH)	503,170	1,364,755	27,268,233
18. Net Electrical Energy (MWH)	478,286	1,294,788	25,824,784
19. Unit Service Factor	99.4	91.4	71.1
20. Unit Availability Factor	99.4	92.0	71.2
*21. Unit Capacity Factor (MDC Net)	96.5	89.0	87.4
22. Unit Capacity Factor (DER Net)	92.8	85.5	64.3
23. Unit Forced Outage Rate	.6	.4	3.2

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
SCHED. MAINT. & INSPEC. - APRIL 26 - JUNE 17, 1980
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
9	03/04/80	F	4.2	B	9		CB	MOTORX	UNIT WAS REMOVED FROM SERVICE DUE TO HIGH OIL LEVEL IN REACTOR COOLANT PUMP MOTOR NO. 4B. CORRECTIVE ACTIONS INCLUDED DRAINING OIL TO NORMAL LEVEL.
10	03/18/80	F	.0	A	5		HC	HTEXCH	LOAD REDUCTION TO REPAIR CONDENSER TUBE LEAK.

SUMMARY

OPERATION WAS AT NEAR FULL POWER DURING THE REPORT PERIOD, EXCEPT FOR AN OUTAGE ON 03/04/80 TO DRAIN OIL FROM THE NO. 4B REACTOR COOLANT PUMP MOTOR. LOAD WAS BRIEFLY REDUCED ON 03/18/80 TO REPAIR A CONDENSER TUBE LEAK.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

VERMONT YANKEE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Vermont/Windham 5 miles S of Brattleboro
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	1593
5.	Design Electrical Rating (MWE Net)	514
6.	Date of Initial Criticality	March 24, 1972
7.	Date of Commercial Operation	November 30, 1972
8.	Condenser Cooling Method	Closed Cycle Cooling Towers
9.	Condenser Cooling Water Source	Connecticut River

Utility & Contractor Information

10.	Licensee	Vermont Yankee Nuclear Power Corp.
11.	Corporate Address	20 Turnpike Road Westboro, Massachusetts 01581
12.	Corporate Contact	D. W. Edwards Assistant to the Vice President
13.	Architect/Engineer	Ebasco
14.	Nuclear Steam Supply System	GE
15.	Constructor	Ebasco

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	IE Senior Resident	W. Raymond
19.	IE Resident Inspector	S. Collins
20.	Licensing Project Manager	V. L. Rooney
21.	Docket Number	50-271
22.	License Number/Date of Issue	DPR-28, 3/21/72
23.	Public Document Room	Brooks Memorial Library 224 Main Street Brattleboro, Vermont 05301

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

FACILITY ITEMS (PLANS AND PROCEDURES):

- Unit was shutdown April 1st and 2nd for a preplanned outage to facilitate a rod program change, startup transformer tap change as identified in LER 80-11/1T, and for routine maintenance.
- Next scheduled outage is for refueling operations in September.

PLANT STATUS:

- Routine operations at full power. SJAE offgas release rate is 3330 microcuries per second. Stack release rate is below 100 microcuries per second.

- LAST IE SITE INSPECTION DATE: 3/14/80
- INSPECTION REPORT NO: 50-271/80-04

SUMMARY

INSPECTION SUMMARY

- 50-271/79-14 - Oct 1-4: Routine, unannounced inspection by a region-based inspecter of radiation protection during refueling, including licensee action on IE Circulars, procedures, advance planning and preparation, training, exposure control, respiratory protection, posting and control, surveys, and radioactive material contro. Upon arrival, a tour was conducted of the radiation control area to observe radiation safety practices on jobs in progress. The inspection involved 33 inspector-hours onsite by one NRC region-based inspector. Of the nine areas inspected, one item of noncompliance was found in each of two areas: (Failure to adhere to RWP procedure; and, Failure to perform surveys).
- 50-271/79-20 - Nov 19-21: Routine, unannounced inspection by a region-based inspecter of pipe support and restraint system surveillance and the Inservice Inspection Program for class 1 and 2 restraints. The inspection involved 21 inspecter-hours onsite by one NRC region-based inspecter. No items of noncompliance were identified.
- 50-271/80-01 - Jan 18: Management meeting held at NRC request to discuss details related to the establishment of an onsite office for the purpose of placing two full time NRC inspectors at the facility. All parties concurred on administrative and operational considerations needed to establish an onsite NRC resident office.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
80-007/03L	01/28/80	02/27/80	30 Day LER	HPCI system declared inoperable during surveillance testing when steam turbine governor valve failed to automatically open following turbine shutdown
80-008/03L	02/03/80	03/03/80	30 Day LER	Snubber RR-2 found inoperable during routine inspections
80-009/03L	02/23/80	03/17/80	30 Day LER	RHR service water pump failed to start during surveillance due to faulty circuit breaker
80-011/01T	03/12/80	03/26/80	24 Hours	Possible degradation of voltage to safeguards equipment

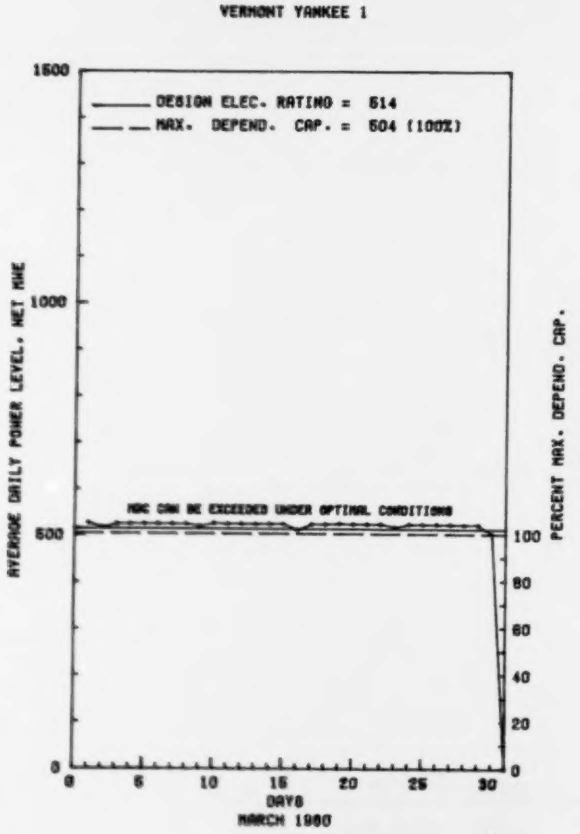
- 50-271/80-02 - Feb 5-8: Routine, unannounced inspection by a region-based inspector of Licensee Actions for IEB 80-01; plant modifications resulting from NUREG 0578; Plant Operations; Emergency Procedure conformance to NEDO-24708 - Small Break LOCA; Operator Guidelines; and submitted LER information. The inspection involved 16 inspector-hours onsite. No items of noncompliance were identified.
- 50-271/80-03 - Feb 21: Special, unannounced inspection by a region-based inspector of the circumstances surrounding a shipment of low specific activity (LSA) radioactive waste on a tractor-trailer which had radiation levels above the DOT limits given in 49 CFR 173.393(j), when the tractor-trailer arrived at the Nuclear Engineering Company (NECO), Richland, Washington, burial site. Another region-based inspector discussed the reporting of significant events with the licensee. The inspection involved 8 inspector-hours onsite by two NRC region-based inspectors. One apparent item of noncompliance was identified in the transportation of waste (Violation - Failure to maintain radiation levels at the underneath surface of an exclusive use trailer within the limits given in 49 CFR 173.393(j)(2)).

REVIEWED BY OMPA LOEB: John J. [illegible] **DATE:** 5-12-80

REVIEWED BY NRR DOR: **DATE:** 5/12/80

VERMONT YANKEE

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-271
2. Utility Contact: F.C. BEERS (617) 366-9011
3. Licensed Thermal Power (MWt): 1593
4. Nameplate Rating (Gross MWe): 626 x 0.9 = 563
5. Design Electrical Rating (Net MWe): 514
6. Maximum Dependable Capacity (Gross MWe): 535
7. Maximum Dependable Capacity (Net MWe): 504
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:

NONE

9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	2,184.0	65,978.8
12. Hours Reactor Was Critical	721.5	2,056.2	53,214.8
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	721.5	2,032.1	51,359.9
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,162,171	3,108,136	72,332,747
17. Gross Electrical Energy (MWH)	392,664	1,065,240	24,034,910
18. Net Electrical Energy (MWH)	376,988	1,022,639	22,790,806
19. Unit Service Factor	97.0	93.0	77.8
20. Unit Availability Factor	97.0	93.0	77.8
21. Unit Capacity Factor (MDC Net)	100.5	92.9	68.5
22. Unit Capacity Factor (DER Net)	98.6	91.1	67.2
23. Unit Forced Outage Rate	.0	1.3	6.3

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

NONE

25. If Shut Down At End of Report Period, Estimated Date of Startup: 04/02/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-9	03/30/80	S	22.5	B	1		HD	VALVEX	MAIN STEAM VALVE WHICH ISOLATES THE STEAM SUPPLY TO THE MAIN TURBINE STEAM SEAL REGULATOR WAS LEAKING STEAM TO THE ATMOSPHERE. VALVE WAS REPLACED WITH PIPING.

SUMMARY

OPERATION WAS AT NEAR FULL POWER DURING THE REPORT PERIOD, WITH THE UNIT TAKEN OFF LINE ON 03/30/80 TO REPAIR A LEAKING STEAM VALVE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

YANKEE ROWE

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Massachusetts/Franklin 25 miles NE of Pittsfield
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	600
5.	Design Electrical Rating (MWE Net)	175
6.	Date of Initial Criticality	August 19, 1960
7.	Date of Commercial Operation	July 1961
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Deerfield River

Utility & Contractor Information

10.	Licensee	Yankee Atomic Electric Company
11.	Corporate Address	20 Turnpike Road Westboro, Massachusetts 01581
12.	Corporate Contact	D. W. Edwards, Asst. to V.P.
13.	Architect/Engineer	Stone & Webster
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Senior Inspector	R. R. Keimig
19.	IE Principal Inspector	W. J. Lazarus
20.	Licensing Project Manager	A. Burger
21.	Docket Number	50-29
22.	License Number/Date of Issue	DPR-3, 12/24/63
23.	Public Document Room	Greenfield Community College 1 College Drive Greenfield, Massachusetts 01301

INSPECTION STATUS

ENFORCEMENT STATUS

FAILURE TO FOLLOW PACKAGE SEARCH REQUIREMENTS.
(8001 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

- The LP Turbine has failed (thrown blades and damaged inner casing).

FACILITY ITEMS (PLANS AND PROCEDURES):

- Discussions with Plant Superintendent indicate that the Turbine is repairable and the plant should be able to recommence power operations in about six months.

MANAGERIAL ITEMS:

- NRR Lessons Learned Task Force met with Yankee Atomic Electric Co. representatives on April 2, 1980 to discuss Yankee Rowe's implementation of Category "A" items.

PLANT STATUS:

- Plant shutdown/cooled down while repairing LP turbine.

- LAST IE SITE INSPECTION DATE: 3/17/80

- INSPECTION REPORT NO: 50-29/80-07

SUMMARY

INSPECTION SUMMARY

- 50-29/79-15 - Dec 13-14: Special unannounced inspection by a NRC region-based inspector of licensee's response to IE Bulletin 79-19 including a review of copies of DOT and NRC regulations maintained onsite, procedures, training of personnel, records, audits, and Quality Assurance relating to the transfer, packaging and shipping of low level radioactive waste material. The inspection involved 16 hours onsite by one NRC region0based inspector. No items of noncompliance were identified.

- 50-29/80-01 - Jan 15-17: Routine, unannounced inspection of the environmental monitoring programs for operation. Areas inspected included: The management controls for these programs, quality controls of analytical measurements, implementation of radiological environmental monitoring programs, security-package search, and a followup on licensee action on previous environmental inspection findings. This inspection involved 48 inspector-hours onsite by 2 region-based NRC inspectors. Of the 5 areas inspected, no items of noncompliance were found in 4 areas. One apparent item of noncompliance was identified in 1 area (Infraction - Failure to observe search requirements).

50-29/80-05 - Feb 19-20: Special unannounced inspection by a region-based inspector of circumstances surrounding a transport trailer carrying Yankee Atomic Electric Company LSA radioactive waste having surface radiation levels greater than 200 mR/hr, when the trailer was received at the Chem-Nuclear Systems, Inc. burial facility near Barnwell, South Carolina. The inspection involved 9 inspector-hours onsite by one region-based inspector. No items of noncompliance or deviations were identified.

50-29/80-06 - Mar 4-6: Facility Organization and Operation, Measurement and Controls, Shipping and Receiving, Storage and Internal Controls, Inventory, Records and Reports and Management and Material Control. The inspection involved 32 inspector-hours onsite by two NRC inspectors and was begun during the regular hours. The licensee was found to be in compliance with NRC requirements in all areas examined during the inspection.

REPORTS RECEIVED FROM LICENSEE

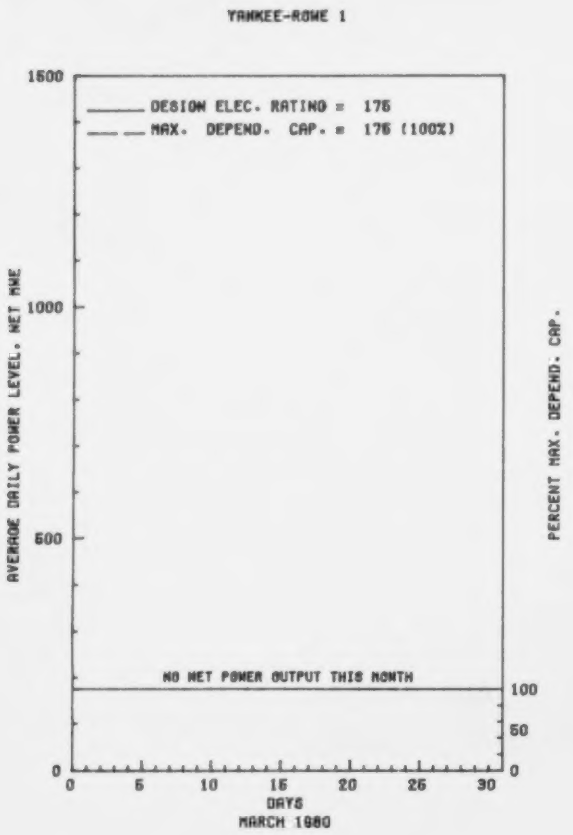
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
80-006/03L	01/26/80	02/25/80	30 Day LER	SI accumulator safety valve SI-SV-609 failed to operate
80-007/03L	01/28/80	02/27/80	30 Day LER	Failure of SI level switch in N2 accu

REVIEWED BY OMPA LOEB: **DATE:** 5/14/80

REVIEWED BY NRR DOR: L. Burger **DATE:** 5/13/80

YANKEE ROWE

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-029
2. Utility Contact: F.C. BEERS (617) 366-9011
3. Licensed Thermal Power (MWt): 600
4. Nameplate Rating (Gross MWe): 185 x 1.0 = 185
5. Design Electrical Rating (Net MWe): 175
6. Maximum Dependable Capacity (Gross MWe): 180
7. Maximum Dependable Capacity (Net MWe): 175
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

CAPACITY FACTORS CALCULATED WITH A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	2,184.0	169,821.0
12. Hours Reactor Was Critical	.0	445.8	138,810.4
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	.0	439.7	134,467.1
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	253,769	72,002,091
17. Gross Electrical Energy (MWH)	0	80,635	22,152,788
18. Net Electrical Energy (MWH)	0	75,749	20,741,893
19. Unit Service Factor	.0	20.1	79.2
20. Unit Availability Factor	.0	20.1	79.2
21. Unit Capacity Factor (MDC Net)	.0	19.8	72.0
22. Unit Capacity Factor (DER Net)	.0	19.8	72.0
23. Unit Forced Outage Rate	100.0	76.6	2.6

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 09/15/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
1	01/19/80	F	744.0	A	4				SHUTDOWN FOR TURBINE ROTOR INVESTIGATION AND REPAIR CONTINUES.

SUMMARY

THE UNIT REMAINED SHUTDOWN THROUGHOUT THE REPORT PERIOD DUE TO TURBINE DISC FAILURE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

ZION 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Illinois/Lake 40 miles N of Chicago
2.	Electric Reliability Council	Mid America Interpool Network
3.	Type of Reactor	PWR
4.	Power Level (MWT)	3250
5.	Design Electrical Rating (MWE Net)	1040
6.	Date of Initial Criticality	June 19, 1973
7.	Date of Commercial Operation	December 31, 1973
8.	Condenser Cooling Method	Straight Through
9.	Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10.	Licensee	Commonwealth Edison Co.
11.	Corporate Address	P. O. Box 767 Chicago, Illinois
12.	Corporate Contact	D. L. Peoples
13.	Architect/Engineer	Sargent & Lundy
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Various

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Senior Inspector	R. Spessard
19.	IE Resident Inspector.	J. E. Kohler (312) 746-2313
20.	Licensing Project Manager	E. Reeves
21.	Docket Number	50-295
22.	License Number/Date of Issue	DPR-39 issued 4/6/73
23.	Public Document Room	Zion-Benton Public Library 2600 Emmons Avenue Zion, Illinois 60099

INSPECTION STATUS

ENFORCEMENT STATUS

TECHNICAL SPECIFICATION 6.2.A REQUIRES ADHERENCE TO APPROVED PROCEDURES. CONTRARY TO THE ABOVE, ON JANUARY 8, 1980, ZION ADMINISTRATIVE PROCEDURE 10-52-3 WAS NOT FOLLOWED FOR UNIT 1 FOR THE SHIFT TURNOVER BETWEEN SHIFT NUMBER 1 AND SHIFT NUMBER 2. (8001 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

+ Unit 1 operating routinely.

+LAST IE SITE INSPECTION DATE: January 3 through February 6, 1980

+ INSPECTION REPORT NO: 80-01

SUMMARY

INSPECTION SUMMARY

+ Inspection on January 3 through February 6, (80-01): Routine, resident inspection of maintenance, plant operations, security, review of licensee events, IE bulletin and circular review, surveillance, new fuel inspection, reactor power distribution limits, thermal power evaluation, axial flux determination. The inspection involved 159 inspector hours onsite by one NRC inspector. Of the areas inspected, one item of noncompliance was identified. (failure to follow procedures during shift turnover).

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

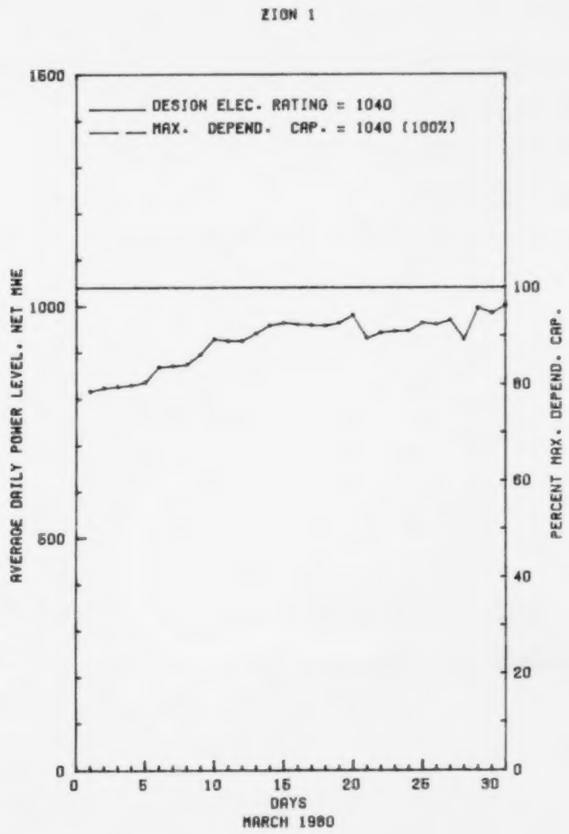
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
80-01/ 03L-0	01/03/80	02/01/80	30 Day LER	While in cold shut down, the spent fuel building area monitor failed full-scale high.
80-09/ 03L-0	02/12/80	03/13/80	30 Day LER	ARM 005 due to defective detector tube.

REVIEWED BY OMPA LOEB: P E Bobe DATE: 5/12/80

REVIEWED BY NRR DOR: E A. Reeves DATE: 5-13-80

ZION 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-295
2. Utility Contact: J. COOK (312) 746-2084
3. Licensed Thermal Power (MWt): 3250
4. Nameplate Rating (Gross MWe): 1220 x 0.9 = 1098
5. Design Electrical Rating (Net MWe): 1040
6. Maximum Dependable Capacity (Gross MWe): 1085
7. Maximum Dependable Capacity (Net MWe): 1040
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	2,184.0	54,792.0
12. Hours Reactor Was Critical	744.0	1,060.7	38,168.4
13. Reactor Reserve Shutdown Hours	.0	.0	2,621.8
14. Hours Generator On-Line	744.0	968.9	37,056.2
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	2,287,740	2,714,595	102,097,596
17. Gross Electrical Energy (MWH)	723,615	850,520	33,082,570
18. Net Electrical Energy (MWH)	688,441	789,682	31,273,928
19. Unit Service Factor	100.0	44.4	67.6
20. Unit Availability Factor	100.0	44.4	67.6
21. Unit Capacity Factor (MDC Net)	89.0	34.8	54.9
22. Unit Capacity Factor (DER Net)	89.0	34.8	54.9
23. Unit Forced Outage Rate	.0	55.6	15.2

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	NONE								

SUMMARY

OPERATION WAS ROUTINE THROUGHOUT THE PERIOD IN POWER ASCENSION FOLLOWING REFUELING, WITH NO OUTAGES OR REDUCTIONS REPORTED.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

ZION 2

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Illinois/Lake 40 miles N of Chicago
2.	Electric Reliability Council	Mid-America Interpool Network
3.	Type of Reactor	PWR
4.	Licensed Power Level (MWT)	3250
5.	Design Electrical Rating (MWE Net)	1040
6.	Date of Initial Criticality	December 24, 1973
7.	Date of Commercial Operation	September 17, 1974
8.	Condenser Cooling Method	Straight Through
9.	Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10.	Licensee	Commonwealth Edison Co.
11.	Corporate Address	P. O. Box 767 Chicago, Illinois
12.	Corporate Contact	D. L. Peoples, Manager of Nuclear Licensing
13.	Architect/Engineer	Sargent & Lundy
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Various

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Senior Inspector	R. Spessard
19.	IE Resident Inspector	J. E. Kohler (312) 746-2313
20.	Licensing Project Manager	E. Reeves
21.	Docket Number	50-304
22.	License Number/Date of Issue	DPR-48, 11/14/73
23.	Public Document Room	Zion-Benton Public Library 2600 Emmans Avenue Zion, Illinois 60099

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

Unit operating routinely.

+LAST IE SITE INSPECTION DATE: January 3 through February 6, 1980

+ INSPECTION REPORT NO: 80-01

SUMMARY

INSPECTION SUMMARY

+ Inspection on January 3 through February 6, (80-01): Routine, resident inspection of maintenance, plant operations, security, review of licensee events, IE bulletin and circular review, surveillance, new fuel inspection, reactor power distribution limits, thermal power evaluation, axial flux determination. The inspection involved 159 inspector hours onsite by one NRC inspector. Of the areas inspected, one item of noncompliance was identified, (failure to follow procedures during shift turnover).

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

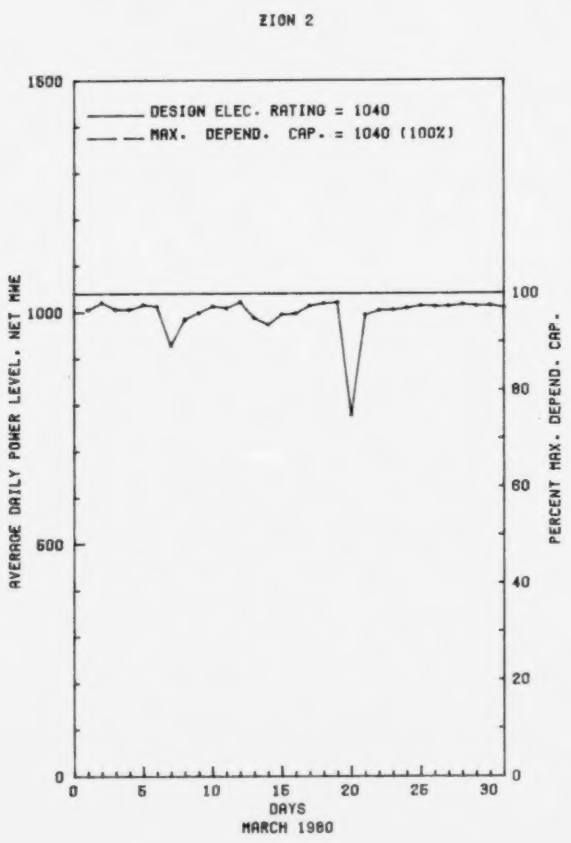
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
80-02/03L-0	01/26/80	02/26/80	30 Day LER	With unit at 98% power, it was noted that pressure was increasing on reference channel.
80-04/03L-0	01/28/80	02/28/80	30 Day LER	During normal operation, found S/G Level xmtr 2LT-538 had shifted high by 3.9%.
80-06/03L-0	02/02/80	03/03/80	30 Day LER	Containment vent particulate rad monitor would not respond to its check source nor to manual bugging.
80-08/03L-0	02/06/80	03/07/80	30 Day LER	At 95% power, instrument loop for steam generator pressure was found to be 2.5% out of tolerance.
80-10/03L-0	02/02/80	03/11/80	30 Day LER	No grab sample was drawn from second shift of rad monitor 2 RE-0011 during OOS surveillance.
80-11/03L-0	02/24/80	03/13/80	30 Day LER	While operating at 98% power, monthly surveillance of MSIV was performed 2 days late.

REVIEWED BY OMPA LOEB: Bobe **DATE:** 5/12/80

REVIEWED BY NRR DOR: Ed A. Reeves **DATE:** 5-13-80

ZION 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-304
2. Utility Contact: J. COOK (312) 746-2084
3. Licensed Thermal Power (MWt): 3250
4. Nameplate Rating (Gross MWe): 1220 x 0.9 = 1098
5. Design Electrical Rating (Net MWe): 1040
6. Maximum Dependable Capacity (Gross MWe): 1085
7. Maximum Dependable Capacity (Net MWe): 1040
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	2,184.0	48,505.0
12. Hours Reactor Was Critical	744.0	1,724.6	35,328.2
13. Reactor Reserve Shutdown Hours	.0	.0	226.1
14. Hours Generator On-Line	744.0	1,708.1	34,522.0
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	2,362,652	5,337,506	97,972,824
17. Gross Electrical Energy (MWH)	776,500	1,748,870	31,470,090
18. Net Electrical Energy (MWH)	741,850	1,665,012	29,866,470
19. Unit Service Factor	100.0	78.2	71.2
20. Unit Availability Factor	100.0	78.2	71.2
21. Unit Capacity Factor (MDC Net)	95.9	73.3	59.2
22. Unit Capacity Factor (DER Net)	95.9	73.3	59.2
23. Unit Forced Outage Rate	.0	21.8	17.9

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUELING - MAY 3, 1980 - APPROX. 7 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A.

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
4	03/06/80	F	.0	A	5				UNIT RAMPED DOWN TO 50% TO ADD OIL TO 2A REACTOR COOLANT PUMP; AND TURBINE STOP VALVE TESTING.
5	03/20/80	F	.0	A	5				UNIT RAMPED DOWN TO 55% TO ADD OIL TO 2A REACTOR COOLANT PUMP; AND TURBINE STOP VALVE TESTING.

SUMMARY

OPERATION WAS AT NEAR FULL POWER DURING THE REPORT PERIOD, WITH LOAD REDUCTIONS ON 03/06/80 AND 03/20/80 FOR TURBINE STOP VALVE TESTING AND TO ADD OIL TO THE NO. 2A REACTOR COOLANT PUMP.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

SECTION 3 – APPENDIX

STATUS OF SPENT FUEL STORAGE CAPABILITY

DATA AS OF 3-31-80

Facility	Core Size [a] (No. of Assemblies)	Present Authorized Storage Pool Capacity (Fuel Assemblies)	No. of Assemblies Stored	Remaining Capacity (No. of Assemblies)	Remaining Capacity If Pending Request Approved (No. of Assemblies)	Scheduled Date For Next Refueling	Will Fill Present Authorized Capacity [b]
I. Pressurized Water Reactors							
ARKANSAS 1	177	590	176	414		12-80	1988
ARKANSAS 2 (d)	177	486	58			03-81	
BEAVER VALLEY 1	157	833	0	833		N/S	1995
CALVERT CLIFFS 1	217	1056 (c)	364 (c)	692 (c)		10-80	1984
CALVERT CLIFFS 2	217					01-81	
COOK 1	193	500	193	307	1857	N/S	1985
COOK 2	193					N/S	
CRYSTAL RIVER 3	177	256	60	196	1109	N/S	1982
DAVIS-BESSE 1	177	735	0	735		04-80	1989
FARLEY 1	157	675	0	675		N/S	1990
FORT CALHOUN 1	133	483	197	286		03-81	1985
GINNA	121	595	156	439		N/S	1987
HADDAM NECK	157	1168	340	828		05-80	1994
INDIAN POINT 1	0	288	160	128		N/S	
INDIAN POINT 2	193	482	200	282		N/S	1984
INDIAN POINT 3	193	837	64	773		N/S	1989
KEWAUNEE	121	990	120	870		05-80	1999
MAINE YANKEE	217	953	433	520		N/S	1986
MILLSTONE 2	217	667	144	523		07-80	1987
NORTH ANNA 1	157	400	0	400	966	N/S	1984
NORTH ANNA 2 (d)	157	400	0	400		N/S	
OCONEE 1	177	1224 (l)	545	(l)	679	02-81	1986
OCONEE 2	177					N/S	
OCONEE 3	177					12-80	
PALISADES	204	798	273	525		N/S	1985
POINT BEACH 1	121	1502 (c)	212 (c)	1290 (c)		N/S	1995
POINT BEACH 2	121					04-80	1995
PRAIRIE ISLAND 1	121	687 (c)	280 (c)	407 (c)		06-80	1984
PRAIRIE ISLAND 2	121					02-81	1984
RANCHO SECO 1	177	579	151	428		07-81	1987
ROBINSON 2	157	272	99	173 (e)		06-80	1980 (g)
SALEM 1	193	264	40	224	1170	09-80	1982
SAN ONOFRE 1	157	216	45	171		04-80	1983
ST LUCIE 1	217	728	60	668		N/S	1990
SURRY 1	157	1044 (c)	1983 (c)	608 (c)		04-80	1NOW
SURRY 2	157					N/S	
THREE MILE ISLAND 1	177	752	208	544		N/S	1989
THREE MILE ISLAND 2	177	442	0	442		N/S	1986
TROJAN	193	651	64	587		04-80	1988
TURKEY POINT 3	157	621	346 (c)	275 (c)		N/S	1987
TURKEY POINT 4	157					N/S	1987
YANKEE-ROWE 1	76	391	149	242	572	N/S	1985
ZION 1	193	868 (c)	436 (c)	432 (c)	1676	01-81	1983
ZION 2	193					05-80	1983

Facility	Core Size [a] (No. of Assemblies)	Present Authorized Storage Pool Capacity (Fuel Assemblies)	No. of Assemblies Stored	Remaining Capacity (No. of Assemblies)	Remaining Capacity If Pending Request Approved (No. of Assemblies)	Scheduled Date For Next Refueling	Will Fill Present Authorized Capacity [b]
II. Boiling Water Reactors							
BIG ROCK POINT 1	84	193	88	105	353	10-80	1983
BROWNS FERRY 1	764	3471	576	2795		04-81	1985
BROWNS FERRY 2	764	1080	400	680		09-80	1985
BROWNS FERRY 3	764	1528	352	1176		10-80	1985
BRUNSWICK 1	560	(f)	(k)			04-80	1986
BRUNSWICK 2	560	(f)	91 (PWR) + 132 (BWR)			N/S	1986
COOPER STATION	548	2366	284	2082		N/S	1995
DRESDEN 1	464	672	221	451		N/S	1986
DRESDEN 2	724	2840 (c)	1069 (c)	1771 (c)	6491 (c)	N/S	1985
DRESDEN 3	724					N/S	1984
DUANE ARNOLD	368	2050	364	1686		04-81	1998
FITZPATRICK	560	760	268	492	1752	05-80	1981
HATCH 1	560	840	260	580		N/S	1986
HATCH 2	560	1120	0	1120		N/S	
HUMBOLDT BAY	172	487	251	236		N/S	
LA CROSSE	72	134	113	21	327	08-80	1NOW
MILLSTONE 1	580	2184	776	1408		09-80	1991
MONTICELLO	484	2237	828	1409		N/S	1991
NINE MILE POINT 1	532	1984	660	1324	2349	N/S	1990
OYSTER CREEK 1	560	1800	780	1020		N/S	1987
PEACH BOTTOM 2	764	2816	618	2198		N/S	1990
PEACH BOTTOM 3	764	2816	712	2104		03-81	1991
PILGRIM 1	580	2320	764	1556		09-81	1990
QUAD CITIES 1	724	1460	596	864		09-80	1986
QUAD CITIES 2	724	1460	492	968		N/S	1985
VERMONT YANKEE 1	368	2000	894	1106		N/S	1988
III. Independent Spent Fuel Storage Installations (h)							
Morris Operations		750 MTU (j)	315	385 MTU (j)	1490 MTU (j)		
NFS(i)		250 MTU	170 MTU	80 MTU			

(a) At each refueling outage approximately 1/3 of a PWR core and 1/4 of a BWR core is off-loaded.
(b) Some of these dates have been adjusted by staff assumptions.
(c) This is the total for both units.
(d) Plant not in commercial operation.
(e) Some spent fuel stored at Brunswick.
(f) Authorized a total 2088 BWR and 304 PWR assemblies for both pools.
(g) Robinson 2 assemblies being shipped to Brunswick for storage.
(h) Capacity is in metric tons of uranium; 1 MTU = 2 PWR assemblies or 5 BWR assemblies.
(i) No longer accepting spent fuel.
(j) Racked for 700 MTU.
(k) Reserved
(l) This is the station total.

N/S = Not Scheduled

REACTOR-YEARS OF EXPERIENCE

AS OF 03/31/80

The total of Reactor years of experience is as the sum of all calendar days for each unit, from the date that electricity was first generated until a final shutdown date or the status date, whichever comes first, divided by 365.25 days/year.

If a date is unknown, the first day of the first month of operation is substituted. Units which have not yet generated electricity but which are licensed are listed but not included in the computation.

(INFORMATION INCLUDES BOTH LICENSED AND NON–LICENSED ELECTRICAL UNITS).

LICENSED OPERATING ELECTRICAL PRODUCING UNITS

UNIT NAME	FIRST ELEC. GENERATION	YEARS EXPERIENCE
ARKANSAS 1	08/01/74	5.67
ARKANSAS 2	12/26/78	1.18
BEAVER VALLEY 1	06/14/76	3.80
BIG ROCK POINT 1	12/08/62	17.31
BROWNS FERRY 1	10/15/73	6.46
BROWNS FERRY 2	08/28/74	5.59
BROWNS FERRY 3	09/12/76	3.55
BRUNSWICK 1	12/04/76	3.32
BRUNSWICK 2	04/29/75	4.93
CALVERT CLIFFS 1	01/03/75	5.24
CALVERT CLIFFS 2	12/07/76	3.32
COOK 1	02/10/75	5.14
COOK 2	03/22/78	2.03
COOPER STATION	05/10/74	5.89
CRYSTAL RIVER 3	01/30/77	3.17
DAVIS-BESSE 1	08/28/77	2.59
DRESDEN 1	04/15/60	19.96
DRESDEN 2	04/13/70	9.97
DRESDEN 3	07/22/71	8.70
DUANE ARNOLD	05/19/74	5.87
FARLEY 1	08/18/77	2.62
FITZPATRICK	02/01/75	5.16
FORT CALHOUN 1	08/25/73	6.60
FORT ST VRAIN	12/11/76	3.30
GINNA	12/02/69	10.33
HADDAM NECK	08/07/67	12.65
HATCH 1	11/11/74	5.39
HATCH 2	09/22/78	1.52
INDIAN POINT 2	06/26/73	6.77
INDIAN POINT 3	04/27/76	3.93
KEWAUNEE	04/08/74	5.98
LA CROSSE	04/26/68	11.93
MAINE YANKEE	11/08/72	7.39
MILLSTONE 1	11/29/70	9.34
MILLSTONE 2	11/09/75	4.39
MONTICELLO	03/05/71	9.08
NINE MILE POINT 1	11/09/69	10.39
NORTH ANNA 1	04/17/78	1.96
OCONEE 1	05/06/73	6.90
OCONEE 2	12/05/73	6.32
OCONEE 3	09/01/74	5.58
OYSTER CREEK 1	09/23/69	10.52
PALISADES	12/31/71	8.25
PEACH BOTTOM 2	02/18/74	6.12
PEACH BOTTOM 3	09/01/74	5.58
PILGRIM 1	07/19/72	7.70
POINT BEACH 1	11/06/70	9.40
POINT BEACH 2	08/02/72	7.66
PRAIRIE ISLAND 1	12/04/73	6.32
PRAIRIE ISLAND 2	12/21/74	5.28
QUAD CITIES 1	04/12/72	7.97
QUAD CITIES 2	05/23/72	7.86
RANCHO SECO 1	10/13/74	5.47
ROBINSON 2	09/26/70	9.51
SALEM 1	12/25/76	3.27

LICENSED OPERATING ELECTRICAL PRODUCING UNITS - CONT.

UNIT NAME	FIRST ELEC. GENERATION	YEARS EXPERIENCE
SAN ONOFRE 1	07/16/67	12.71
ST LUCIE 1	05/07/76	3.90
SURRY 1	07/04/72	7.74
SURRY 2	03/10/73	7.06
THREE MILE ISLAND 1	06/19/74	5.79
TROJAN	12/23/75	4.27
TURKEY POINT 3	11/02/72	7.41
TURKEY POINT 4	06/21/73	6.78
VERMONT YANKEE 1	09/20/72	7.53
YANKEE-ROWE 1	11/10/60	19.39
ZION 1	06/28/73	6.76
ZION 2	12/26/73	6.26
TOTAL		457.77 YEARS

PERMANENTLY OR INDEFINITELY SHUTDOWN UNITS

UNIT NAME	FIRST ELEC. GENERATION	SHUTDOWN OR ELECTRICAL CESSATION DATE	YEARS EXPERIENCE
BONUS	08/14/64	06/01/68	3.80
CVTR	12/18/63	01/01/67	3.04
ELK RIVER	08/24/63	02/01/68	4.44
FERMI 1	08/05/66	11/29/72	6.32
HALLAM	05/29/63	09/01/64	1.26
HUMBOLDT BAY	04/18/63	07/02/76	13.21
INDIAN POINT 1	09/16/62	10/31/74	12.12
PATHFINDER	07/25/66	10/01/67	1.19
PEACH BOTTOM 1	01/27/67	11/01/74	7.76
PIQUA	11/04/63	01/01/66	2.16
THREE MILE ISLAND 2	04/21/78	03/28/79	.93
TOTAL			56.23 YEARS

NON–POWER REACTORS IN THE U.S.

1. RESEARCH REACTORS

State	Location	Licensee	Reactor Type	Docket	License Number	Date Issued	Authorized Power Level (KW)
Alabama	1. Tuskegee	Tuskegee Institute	AGN 201 No. 102	50-406	R-122	08-30-74	.0001
Arizona	2. Tucson	U. of Arizona	TRIGA Mk. I	50-113	R-52	12-05-58	100
California	3. Los Angeles	U. of California	Argonaut	50-142	R-71	10-03-60	100
	4. Berkeley	U. of California	TRIGA Mark III	50-224	R-101	08-10-66	1000
	5. Irvine	U. of California	TRIGA Mark I	50-326	R-116	11-24-69	250
	6. Santa Barbara	U. of California	L-77	50-433	R-124	12-03-74	.01
	7. Canoga Park	Rockwell International Corporation	L-77	50-94	R-40	05-17-58	.01
	8. Hawthorne	Northrop Corporate Laboratories	TRIGA Mark F	50-187	R-90	03-04-63	1000
	9. San Luis Obispo	California State Polytechnic College	AGN 201 No. 100	50-394	R-121	05-16-73	.0001
	10. Alameda County	General Electric Co.	NTR	50-73	R-33	10-31-57	100
	11. San Diego	General Atomic Co.	TRIGA Mark I	50-89	R-38	05-03-58	250
	12. San Diego	General Atomic Co.	TRIGA Mark F	50-163	R-67	07-01-60	1500
	13. San Ramon	Aerotest Operations, Inc.	TRIGA (Industrial)	50-228	R-98	07-02-65	250
Colorado	14. Jefferson County	U.S. Geological Survey, Dept. of Interior	TRIGA Mark I	50-274	R-113	02-24-69	1000
Delaware	15. Newark	U. of Delaware	AGN-201 No. 113	50-98	R-43	07-03-58	.0001
District of Columbia	16. –	Catholic U.	AGN-201 No. 101	50-77	R-31	11-15-67	.0001
Florida	17. Gainesville	U. of Florida	Argonaut	50-83	R-56	05-21-59	100
Georgia	18. Atlanta	Georgia Institute of Technology	AGN-201 No. 104	50-276	R-111	04-19-68	.0001
	19. Atlanta	Georgia Institute of Technology	Heavy Water	50-160	R-97	12-29-64	5000
Idaho	20. Pocatello	Idaho State U.	AGN-201 No. 103	50-284	R-110	10-11-67	.0001
Illinois	21. Urbana	U. of Illinois	Advanced TRIGA	50-151	R-115	07-22-69	1500
	22. Urbana	U. of Illinois	LOPRA	50-356	R-117	12-27-71	10
	23. Zion	Westinghouse Electric Corporation	NTR	50-87	R-119	01-28-72	10
Indiana	24. West Lafayette	Purdue U.	Lockheed	50-182	R-87	08-16-62	10

RESEARCH REACTORS – CONT.

State	Location	Licensee	Reactor Type	Docket	License Number	Date Issued	Authorized Power Level (KW)
Iowa	25. Ames	Iowa State U.	UTR-10	50-116	R-59	10-16-59	10
Kansas	26. Lawrence	U. of Kansas	Lockheed	50-148	R-78	06-23-61	250
	27. Manhattan	Kansas State U.	TRIGA	50-188	R-88	10-16-62	250
Maryland	28. College Park	U. of Maryland	TRIGA	50-166	R-70	10-14-60	250
	29. Bethesda	Armed Forces Radiobiology Research Institute	TRIGA	50-170	R-84	06-26-62	1000
Massachusetts	30. Lowell	U. of Lowell	GE	50-223	R-125	12-24-74	1000
	31. Cambridge	Massachusetts Institute of Technology	Heavy Water Reflected	50-20	R-37	06-09-58	5000
	32. Worcester	Worcester Polytechnic Institute	GE	50-134	R-61	12-16-59	10
Michigan	33. Ann Arbor	U. of Michigan	Pool	50-2	R-28	09-13-57	2000
	34. Lansing	Michigan State U.	TRIGA Mark I	50-294	R-114	03-21-69	250
	35. Midland	Dow Chemical Co.	TRIGA Mark I	50-264	R-108	07-03-67	100
Missouri	36. Rolla	U. of Missouri	Pool	50-123	R-79	11-21-61	200
	37. Columbia	U. of Missouri	Tank	50-186	R-103	10-11-66	10,000
Nebraska	38. Omaha	V.A. Hospital	TRIGA	50-131	R-57	06-26-59	18
New Mexico	39. Albuquerque	U. of New Mexico	AGN-201M No. 112	50-252	R-102	09-17-66	.005
New York	40. Buffalo	State U. of New York	PULSTAR'	50-57	R-77	03-24-61	2000
	41. Sterling Forest	Union Carbide Corp.	Pool	50-54	R-81	09-07-61	5000
	42. New York City	Columbia U.	TRIGA Mark II	50-208	R-128	04-14-77	250
	43. Ithaca	Cornell U.	ZPR	50-97	R-89	12-11-62	.1
	44. Ithaca	Cornell U.	TRIGA Mk. II	50-157	R-80	01-11-62	100
	45. Riverdale	Manhattan College	Tank	50-199	R-94	03-24-64	.0001
North Carolina	46. Raleigh	North Carolina State U.	PULSTAR	50-297	R-120	08-25-72	1000
Ohio	47. Columbus	Ohio State U.	Pool	50-150	R-75	02-24-61	10
Oklahoma	48. Norman	U. of Oklahoma	AGN-211 No. 102	50-112	R-53	12-29-58	.015
Oregon	49. Corvallis	Oregon State U.	AGN-201 No. 114	50-106	R-51	11-13-59	.0001
	50. Corvallis	Oregon State U.	TRIGA Mark II	50-243	R-106	03-07-67	1000
	51. Portland	Reed Institute	TRIGA Mk. I	50-288	R-112	07-02-68	250
Pennsylvania	52. University Park	Pennsylvania State U.	TRIGA Mk. III	50-5	R-2	07-08-55	1000

NON–POWER REACTORS IN THE U.S.

RESEARCH REACTORS – CONT.

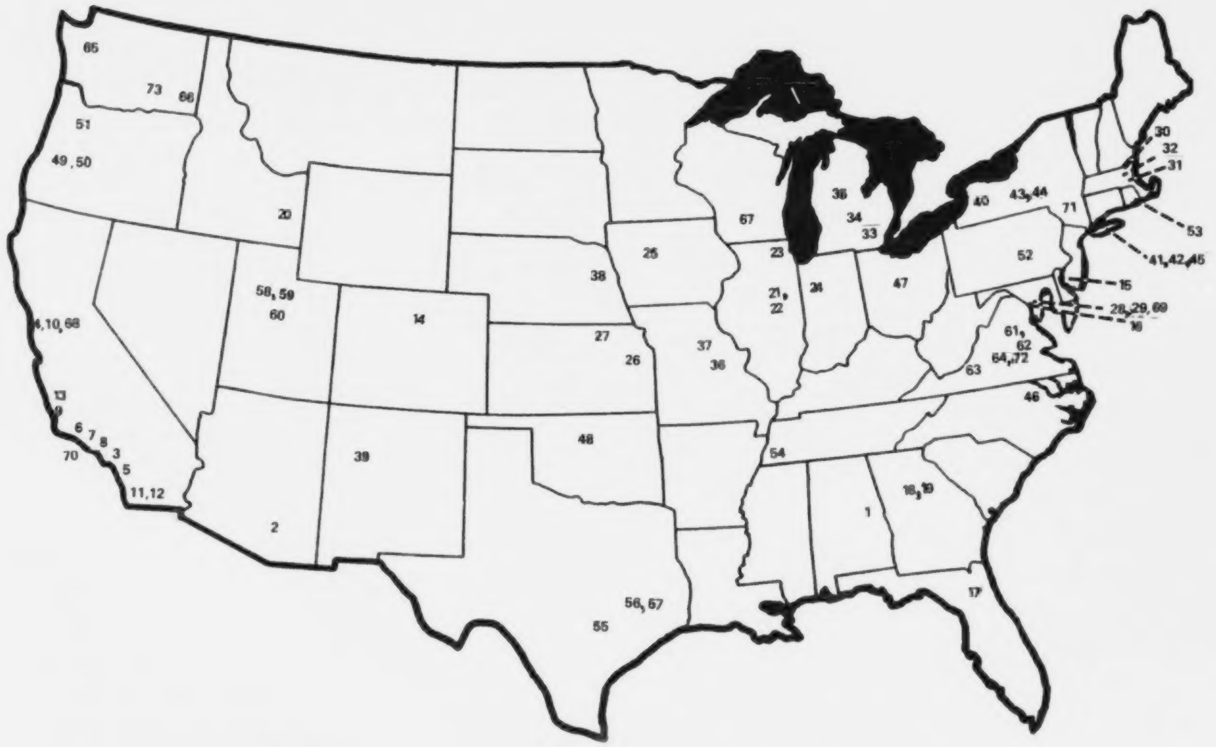
State	Location	Licensee	Reactor Type	Docket	License Number	Date Issued	Authorized Power Level (KW)
Rhode Island	53. Ft. Kearney, Narragansett	Rhode Island & Providence Plantations AEC	GE Pool	50-193	R-95	07-21-64	2000
Tennessee	54. Shelby County	Memphis State U.	AGN-201 No. 108	50-538	R-127	12-10-76	.0001
Texas	55. Austin	U. of Texas	TRIGA Mk. I	50-192	R-92	08-02-63	250
	56. College Station	Texas A&M U.	TRIGA	50-128	R-83	12-07-61	1000
	57. College Station	Texas A&M U.	AGN- 201M No. 106	50-59	R-23	08-26-57	.005
Utah	58. Salt Lake City	U. of Utah	AGN-201M No. 107	50-72	R-25	09-12-57	.005
	59. Salt Lake City	U. of Utah	TRIGA Mark I	50-407	R-126	09-30-75	100
	60. Provo	Brigham Young U.	L-77	50-262	R-109	09-07-67	.01
Virgina	61. Charlottesville	U. of Virgina	Pool	50-62	R-66	06-27-60	2000
	62. Charlottesville	U. of Virginia	Cavalier	50-396	R-123	09-24-74	.1
	63. Blacksburg	Virginia Polytechnic Institute	UTR-10	50-124	R-62	12-18-59	100
	64. Lynchburg	Babcock & Wilcox Co.	LPR	50-99	R-47	09-05-58	1000
Washington	65. Seattle	U. of Washington	Argonaut	50-139	R-73	03-31-61	100
	66. Pullman	Washington State U.	TRIGA	50-27	R-76	03-06-61	1000
Wisconsin	67. Madison	U. of Wisconsin	TRIGA	50-156	R-74	11-23-60	1000

2. EXPERIMENTAL AND TEST REACTORS

State	Location	Licensee	Reactor Type	Docket	License Number	Date Issued	Authorized Power Level (MWt)
California	68. Alameda Co.	General Electric Co.	GETR	50-70	TR-1	01-07-59	50
Maryland	69. Gaithersburg	National Bureau of Standards	TEST	50-184	TR-5	11-13-67 POL 06-30-70 OL	10

3. CRITICAL EXPERIMENT FACILITIES

State	Location	Licensee	Docket	License Number	Date Issued	Experiments
California	70. Ventura Co.	Rockwell International Corp. (Possesion Only)	50-147	CX-17	10-03-60	AETR Experiments (Terminal Requested).
New York	71. Schenectady	Rensselaer Polytechnic Institute	50-225	CX-22	07-03-64	
Virginia	72. Lynchburg	Babcock & Wilcox Co.	50-13	CX-10	10-22-58	NMSR Experiments
Washington	73. Richland	Battelle Memorial Institute	50-360	CX-26	11-29-71	Plutonium Recycle Critical Facility



Source: NRC Facilities License Application Record (FLAR) 06-30-77.

ABBREVIATIONS

AGN	Aerojet General Nuclear	These reactors were constructed by Aeroject at San Ramon, California, and then transferred to the current location.
ZPR	Zero Power Reactor	
LPR	Low-Power Reactor	

DISTRIBUTION LIST—NRC

INTERNAL DISTRIBUTION

Office of the Commissioners	5
Atomic Safety and Licensing Appeal Panel	3
Advisory Committee on Reactor Safeguards	6
Office of Inspector and Auditor	1
Office of Policy Evaluation	1
Office of the General Counsel	1
Office of Public Affairs	21
Office of Congressional Affairs	3
Office of the Executive Director for Operations	7
Office of Administration	2
Office of the Executive Legal Director	2
Office of the Controller	2
Office for Analysis and Evaluation of Operational Data	2
Office of International Programs	3
Office of State Programs	1
Office of Management and Program Analysis	14
Office of Standards Development	1
Office of Nuclear Material Safety and Safeguards	2
— Division of Fuel Cycle and Material Safety	3
— Division of Safeguards	2
Office of Nuclear Reactor Regulation	5
— Division of Project Management	6
— Division of Operating Reactors	105
— Division of Systems Safety	4
— Division of Site Safety and Environmental Analysis	18
Office of Nuclear Regulatory Research	4
Office of Inspection and Enforcement	7
— Region I	11
— Region II	10
— Region III	10
— Region IV	10
— Region V	7
	279

EXTERNAL DISTRIBUTION

SPECIAL REQUESTS

Air Force	1
Congress	10
Department of Energy	32
Government Accounting Office	1
Environmental Protection Agency	1
Electric Power Research Institute	1
Argonne National Laboratory	3
Federal Power Commission	1
Bureau of Mines	1
Department of Agriculture	1
Department of Commerce	1
Department of the Interior	1
	54

OTHER

GPO Depository	440
GPO Stores	75
National Technical Information Service	25
Subscriptions (NTIS)	250
Colleges and Libraries (including Public Document Rooms)	128
Utilities and Other Requests	227
	1145

15/4-5

Nuclear Regulatory Commission

NUREG 0020
VOL. 4 NO. 5

OPERATING UNITS STATUS REPORT

DATA AS OF 4-30-80

MAY 1980

NEWARK PUBLIC LIBRARY
JUL 2 1 1980
U. S. GOVT. PUBS.
DEPOSITORY 376

NEWARK PUBLIC LIBRARY
JUL 2 1 1980
U. S. GOVT. PUBS.
DEPOSITORY 376

LICENSED OPERATING REACTORS

Y3.N88:

Available from
GPO Sales Program
Division of Technical Information and Document Control
U. S. Nuclear Regulatory Commission
Washington, D.C. 20555
and
National Technical Information Service
Springfield., Virginia 22161
703-326-8543
A Year's Subscription, 12 Issues, for this Publication is
$120.00 North American Continent ($240 00 Other).
Multiple Subscriptions Entered at the Same Time to the Same Address
Cost $90.00 North American Continent ($180.00 Other) for each Additional Subscription.

Single copies are $12.50 North American Continent ($25.00 Other).

This document is provided free of charge to those who participate in the data collection program, and to the Nuclear Regulatory Commission's local public document rooms.

Issued by the U.S. Nuclear Regulatory Commission

PUBLISHED: JUNE 19, 1980

Nuclear Regulatory Commission

NUREG 0020
VOL. 4 NO. 5
MAY 1980

OPERATING UNITS STATUS REPORT

DATA AS OF 4-30-80

LICENSED OPERATING REACTORS

AUTHORIZATION AND CLEARANCE

THE U.S. NUCLEAR REGULATORY COMMISSION'S OFFICE OF MANAGEMENT AND PROGRAM ANALYSIS PUBLISHES THIS MONTHLY STATUS REPORT "AS PART OF THE REPORTING REQUIREMENTS IN SECTION 50.36 OF 10 CFR PART 50 UNDER GAO CLEARANCE NUMBER B-180225, WITH AN EXPIRATION DATE OF SEPTEMBER 30, 1981," AS STATED IN THE OCTOBER 3, 1978 LETTER FROM JOHN M. LOVELADY, ASSISTANT DIRECTOR, GENERAL GOVERNMENT DIVISION, U. S. GENERAL ACCOUNTING OFFICE, TO J.M. FELTON, DIRECTOR, DIVISION OF RULES AND RECORDS, U.S. NUCLEAR REGULATORY COMMISSION.

STATEMENT OF PURPOSE

The U. S. Nuclear Regulatory Commission's monthly OPERATING UNITS STATUS REPORT – LICENSED OPERATING REACTORS, provides data on the operation of nuclear units as timely and accurately as possible. This information is collected by the Office of Management and Program Analysis from the Headquarters Staff of NRC's Office of Inspection and Enforcement, from NRC's Regional Offices, and from utilities. Since all of the data concerning operation of the units is provided by the utility operators less than two weeks after the end of the month, necessary corrections to published information are shown on the ERRATA page.

This report is divided into three sections: the first contains monthly highlights and statistics for commercial operating units, and errata from previously reported data; the second is a compilation of detailed information on each unit, provided by NRC Regional Offices, IE Headquarters and the Utilities; and the third section is an appendix for miscellaneous information such as spent fuel storage capability, reactor-years of experience and non-power reactors in the United States.

The percentage computations, Items 19 through 24 in Section 2, the vendor capacity factors on page 1-4, and actual vs. potential energy production on Page 1-2 are computed from actual power production from the period of consideration. The percentages listed in power generation on Page 1-2 are computed as an arithmetical average. The factors for the life-span of each unit (the "Cumulative" column) are reported by the utility and are not entirely re-computed by NRC. Utility power production data is checked for consistency with previously submitted statistics.

In some cases, the numbers in the "Cumulative" column may have been totaled from a date that is different from the date of commercial operation. In these cases, the column is noted with an explanation of the nature of the change on each individual plant page in Section 2.

It is hoped this status report proves informative and helpful to all agencies and individuals interested in analyzing trends in the nuclear industry which might have safety implications, or in maintaining an awareness of the U. S. energy situation as a whole.

PHILIP A. ROSS
REVIEW COORDINATOR

TABLE OF CONTENTS

GLOSSARY ii
INDEX TO LICENSED UNITS iii

SECTION 1 – CURRENT DATA SUMMARIES

MONTHLY HIGHLIGHTS OF COMMERCIAL NUCLEAR POWER UNITS 1-2

- Licensed Power Reactors 1-2
- Power Generation 1-2
- Actual vs. Potential Energy Production 1-2
- Outage Data 1-2
- Derated Units 1-2
- Shutdowns - Number and Reasons 1-3
- Unit Availability, Capacity, and Forced Outage Rate 1-3
- Refueling Schedule 1-3

AVERAGE CAPACITY FACTORS BY VENDOR 1-4

AVERAGE DAILY POWER LEVEL FOR ALL COMMERCIAL OPERATING UNITS 1-5

ERRATA 1-7

SECTION 2 – OPERATING POWER REACTORS

ARKANSAS 1 THROUGH ZION 2 PAGES A–2 TO Z–5

- Facility Data
- Inspection and Enforcement Status
- Reports Received from Licensee
- Operating Status
- Average Daily Power Level (MWe)
- Unit Shutdowns/Reductions

SECTION 3 – APPENDIX

STATUS OF SPENT FUEL STORAGE CAPABILITY 3-2
REACTOR-YEARS OF EXPERIENCE 3-3
NON-POWER REACTORS IN THE U.S. 3-4

GLOSSARY

AVERAGE DAILY POWER LEVEL (MWe)	The net electrical energy generated during the day (measured from 0001 to 2400 hours inclusive) in megawatts hours divided by 24 hours.
LICENSED THERMAL POWER (MWt)	The maximum thermal power of the reactor authorized by the NRC, expressed in megawatts.
DATE OF COMMERCIAL OPERATION	Date unit was declared by utility owner to be available for the regular production of electricity; usually related to satisfactory completion of qualification tests as specified in the purchase contract and to accounting policies and practices of utility.
DESIGN ELECTRICAL RATING (DER) (Net MWe)	The nominal net electrical output of the unit specified by the utility and used for the purpose of plant design.
FORCED OUTAGE	An outage required to be initiated no later than the weekend following discovery of an off-normal condition.
FORCED OUTAGE HOURS	The clock hours during the report period that a unit is unavailable due to forced outages.
GROSS ELECTRICAL ENERGY GENERATED (MWH)	Electrical output of the unit during the report period as measured at the output terminals of the turbine generator, in megawatts hours.
GROSS HOURS	The clock hours from the beginning of a specified situation until its end. For outage durations, the clock hours during which the unit is not in power production.
GROSS THERMAL ENERGY GENERATED (MWH)	The thermal energy produced by the unit during the report period as measured or computed by the licensee in megawatt hours.
HOURS GENERATOR ON-LINE	Also, "Unit Service Hours." The total clock hours in the report period during which the unit operated with breakers closed to the station bus. These hours added to the total outage hours experienced by the unit during the report period, shall equal the hours in the report period.
HOURS IN REPORTING PERIOD	For units in power ascension at the end of the period, the gross hours from the beginning of the period or the first electrical production, whichever comes last, to the end of the period. For units in commercial operation at the end of the period, the gross hours from the beginning of the period or of commercial operation, whichever comes last, to the end of the period or decommissioning, whichever comes first.
HOURS REACTOR CRITICAL	The total clock hours in the report period during which the reactor sustained a controlled chain reaction.
MAXIMUM DEPENDABLE CAPACITY (GROSS) (MDC Gross) (Gross MWe)	Dependable main-unit gross capacity, winter or summer, whichever is smaller. The dependable capacity varies because the unit efficiency varies during the year due to cooling water temperature variations. It is the gross electrical output as measured at the output terminals of the turbine generator during the most restrictive seasonal conditions (usually summer).
MAXIMUM DEPENDABLE CAPACITY (NET) (MDC Net) (Net MWe)	Maximum Dependable Capacity (Gross) less the normal station service loads.
NAMEPLATE RATING (Gross MWe)	The nameplate power designation of the generator in megavolt amperes (MVA) times the nameplate power factor of the generator. NOTE: The nameplate rating of the generator may not be indicative of the maximum or dependable capacity, since some other item of equipment of a lesser rating (e.g., turbine) may limit unit output.
NET ELECTRICAL ENERGY GENERATED	Gross electrical output of the unit measured at the output terminals of the turbine generator during the reporting period, minus the normal station service electrical energy utilization. If this quantity is less than zero, a negative number should be recorded.
OUTAGE	A situation in which no electrical production takes place.
OUTAGE DATE	As reported on Appendix D of Reg. Guide 1.16, the date of the start of the outage. If continued from a previous month, report the same outage date but change "Method of Shutting Down Reactor" to "4 (Other)" and add a note: "Continued from previous month."
OUTAGE DURATION	The total clock hours of the outage measured from the beginning of the report period or the outage, whichever comes last, to the end of the report period or the outage, whichever comes first.
OUTAGE NUMBER	A number unique to the outage assigned by the licensee. The same number is reported each month in which the outage is in progress. One format is "76-05" for the fifth outage to occur in 1976.
PERIOD HOURS	See "Hours in Reporting Period."
POWER REDUCTION	A reduction in the Average Daily Power Level of more than 20% from the previous day. All power reductions are defined as outage of zero hours durations for the purpose of computing unit service and availability factors, and forced outage rate.

REACTOR AVAILABLE HOURS	The total clock hours in the report period during which the reactor was critical or was capable of being made critical. (Reactor Reserve Shutdown Hours + Hours Reactor Critical.)
REACTOR AVAILABILITY FACTOR	$\frac{\text{Reactor Available Hours x 100}}{\text{Period Hours}}$
REACTOR RESERVE SHUTDOWN	The cessation of criticality in the reactor for administrative or other similar reasons when operation could have been continued.
REACTOR RESERVE SHUTDOWN HOURS	The total clock hours in the report period during which the reactor is in reserve shutdown mode. NOTE: No credit is given for NRC imposed shutdowns.
REACTOR SERVICE FACTOR	$\frac{\text{Hours Reactor Critical x 100}}{\text{Period Hours}}$
REPORT PERIOD	Usually, the preceding calendar month. Can also be the preceding calendar year, (Year-to-Date), or the life-span of a unit (cumulative).
RESTRICTED POWER LEVEL	Maximum net electrical generation to which the unit is restricted during the report period due to the state of equipment, external conditions, administrative reasons, or a direction by NRC.
SCHEDULED OUTAGE	Planned removal of a unit from service for refueling, inspection, training, or maintenance. Those outages which do not fit the definition of "Forced Outage" perforce are "Scheduled Outages."
STARTUP AND POWER ASCENSION TEST PHASE	Period following initial criticality during which the unit is tested at successively higher levels, culminating with operation at full power for a sustained period and completion of warranty runs. Following this phase, the utility generally considers the unit to be available for commercial operation.
UNIT	The set of equipment uniquely associated with the reactor, including turbine generators, and ancillary equipment, considered as a single electrical energy production facility.
UNIT AVAILABLE HOURS	The total clock hours in the report period during which the unit operated on-line or was capable of such operation. (Unit Reserve Shutdown Hours + Hours Generator On-Line.)
UNIT AVAILABILITY FACTOR•	$\frac{\text{Unit Available Hours x 100}}{\text{Period Hours}}$
UNIT CAPACITY FACTORS	
– Using Licensed Thermal Power	$\frac{\text{Gross Thermal Energy Generated x 100}}{\text{Period Hours x Lic. Thermal Power}}$
– Using Nameplate Rating	$\frac{\text{Gross Electrical Energy Generated x 100}}{\text{Period Hours x Nameplate Rating}}$
– Using DER	$\frac{\text{Net Electrical Energy Generated x 100}}{\text{Period Hours x DER}}$
– Using MDC Gross	$\frac{\text{Gross Electrical Energy Generated x 100}}{\text{Period Hours x MDC Gross}}$
– Using MDC Net	$\frac{\text{Net Electrical Energy Generated x 100}}{\text{Period Hours x MDC Net}}$

NOTE: if MDC Gross and/or MDC Net have not been determined, the DER is substituted for this quantity for Unit Capacity Factor calculations.

UNIT FORCED OUTAGE RATE	$\frac{\text{Forced Outage Hours}}{\text{Unit Service Hours + Forced Outage Hours}}$
UNIT RESERVE SHUTDOWN	The removal of the unit from on-line operation for economic or other similar reasons when operation could have been continued.
UNIT RESERVE SHUTDOWN HOURS	The total clock hours in the report period during which the unit was in reserve shutdown mode.
UNIT SERVICE FACTOR	$\frac{\text{Unit Service Hours x 100}}{\text{Period Hours}}$
UNIT SERVICE HOURS	See "Hours Generator On-Line."

INDEX TO LICENSED UNITS

ARKANSAS 1	A-3
ARKANSAS 2	A-5
BEAVER VALLEY 1	B-3
BIG ROCK POINT 1	B-5
BROWNS FERRY 1	B-7
BROWNS FERRY 2	B-9
BROWNS FERRY 3	B-11
BRUNSWICK 1	B-13
BRUNSWICK 2	B-15
CALVERT CLIFFS 1	C-3
CALVERT CLIFFS 2	C-5
COOK 1	C-7
COOK 2	C-9
COOPER STATION	C-11
CRYSTAL RIVER 3	C-13
DAVIS-BESSE 1	D-3
DRESDEN 1	D-5
DRESDEN 2	D-7
DRESDEN 3	D-9
DUANE ARNOLD	D-11
FARLEY 1	F-3
FITZPATRICK	F-5
FORT CALHOUN 1	F-7
FORT ST VRAIN	F-9
GINNA	G-3
HADDAM NECK	H-3
HATCH 1	H-5
HATCH 2	H-7
HUMBOLDT BAY	H-9
INDIAN POINT 1	I-3
INDIAN POINT 2	I-5
INDIAN POINT 3	I-7
KEWAUNEE	K-3
LA CROSSE	L-3
MAINE YANKEE	M-3
MILLSTONE 1	M-5
MILLSTONE 2	M-7
MONTICELLO	M-9
NINE MILE POINT 1	N-3
NORTH ANNA 1	N-5
OCONEE 1	O-3
OCONEE 2	O-5
OCONEE 3	O-7
OYSTER CREEK 1	O-9
PALISADES	P-3
PEACH BOTTOM 2	P-5
PEACH BOTTOM 3	P-7
PILGRIM 1	P-9
POINT BEACH 1	P-11
POINT BEACH 2	P-13
PRAIRIE ISLAND 1	P-15
PRAIRIE ISLAND 2	P-17
QUAD CITIES 1	Q-3
QUAD CITIES 2	Q-5
RANCHO SECO 1	R-3
ROBINSON 2	R-5
SALEM 1	S-3
SAN ONOFRE 1	S-5
ST LUCIE 1	S-7
SURRY 1	S-9
SURRY 2	S-11
THREE MILE ISLAND 1	T-3
THREE MILE ISLAND 2	T-5
TROJAN	T-7
TURKEY POINT 3	T-9
TURKEY POINT 4	T-11
VERMONT YANKEE 1	V-3
YANKEE-ROWE 1	Y-3
ZION 1	Z-3
ZION 2	Z-5

SECTION 1 – CURRENT DATA SUMMARIES

MONTHLY HIGHLIGHTS OF COMMERCIAL NUCLEAR POWER UNITS

FOR MONTH OF

APRIL 1980

LICENSED POWER REACTORS:

NUMBER		CAPACITY [1] MWe (Net)
67	IN COMMERCIAL OPERATION	49,010
67 [2]	LICENSED TO OPERATE	49,010
3 [3]	LICENSED FOR FUEL LOADING AND LOW POWER TESTING	

POWER GENERATION	REPORT MONTH	PREVIOUS MONTH	YEAR-TO-DATE
1. GROSS ELECTRICAL (MWHE)	19,135,853	20,881,056	80,343,171
2. NET ELECTRICAL (MWHE)	18,263,263	19,914,705	76,957,756
3. AVG. UNIT SERVICE FACTOR (%)	56.2	59.3	59,8
4. AVERAGE UNIT AVAILABILITY FACTOR (%)	55.5	59.6	60.2
5. AVERAGE UNIT CAPACITY FACTOR (MDC) (%)	49.2	53.9	53.5
6. AVG. UNIT CAPACITY FACTOR (DER) (%)	48.1	52.6	52.2
7. FORCED OUTAGE RATE (%)	11.5	11.6	10.7

NOTES

[1] BASED UPON MAXIMUM DEPENDABLE CAPACITY; DESIGN ELECTRIC RATING USED IF MDC NOT DETERMINED.

[2] EXCLUDES INDIAN POINT 1 (DER = 265), HUMBOLDT BAY (DER = 65), AND THREE MILE ISLAND 2 (DER = 906) WHICH HAVE OPERATING LICENSES BUT ARE SHUTDOWN INDEFINITELY.

[3] SEQUOYAH 1 (DER =1148) WAS LICENSED ON FEBRUARY 29, 1980
NORTH ANNA 2 (DER = 907) WAS LICENSED ON APRIL 11, 1980
SALEM 2 (DER =1115) WAS LICENSED ON APRIL 18, 1980

ACTUAL VS. POTENTIAL ENERGY PRODUCTION

			% OF POTENTIAL PRODUCTION
POTENTIAL ENERGY PRODUCTION IN THIS PERIOD BY UNITS IN COMMERCIAL OPERATION (Using Maximum Dependable Capacity Net)		35,238,190 MWHe	
ENERGY ACTUALLY PRODUCED DURING THIS REPORT PERIOD	=	18,263,263 MWHe NET	51.8
ENERGY NOT PRODUCED DURING THIS REPORT PERIOD DUE TO SCHEDULED OUTAGES (NET)	=	11,605,975 MWHe	32.9
ENERGY NOT PRODUCED DURING THIS REPORT PERIOD DUE TO FORCED OUTAGES (NET)	=	3,228,305 MWHe	9.2
ENERGY NOT PRODUCED DURING THIS REPORT PERIOD FOR OTHER REASONS (NET)	=	2,140,647 MWHe	6.1
TOTAL		35,238,190 MWHe =	100%
ENERGY NOT PRODUCED THIS REPORT PERIOD DUE TO NRC IMPOSED SHUTDOWNS AND RESTRICTIONS	=	3,260,813 MWHe	9.3
NUMBER OF UNITS WITH POWER RESTRICTIONS IMPOSED BY NRC	=	1	

OUTAGE DATA

	NUMBER	HOURS	PERCENT OF CLOCK TIME	MWHE LOST PRODUCTION
1. FORCED OUTAGES DURING MONTH	57	4,662.1	9.7	3,228,305
2. SCHEDULED OUTAGES DURING THE MONTH	39	16,898.0	35.1	11,605,975
3. TOTAL	96	21,560.1	44.8	14,834,280

MWHE LOST PRODUCTION = Down time x maximum dependable capacity net

DERATED UNITS

UNIT NAME	MAXIMUM DEPENDABLE CAPACITY (MWe - NET)	POWER LIMIT (MWe - NET)
BIG ROCK POINT 1	65	63
†FORT ST. VRAIN	330	231
HADDAM NECK	550	484
INDIAN POINT 2	849	824
POINT BEACH 1	495	390
ROBINSON 2	665	665

†Restricted by NRC.

MONTHLY HIGHLIGHTS OF COMMERCIAL NUCLEAR POWER UNITS

FOR MONTH OF

APRIL 1980

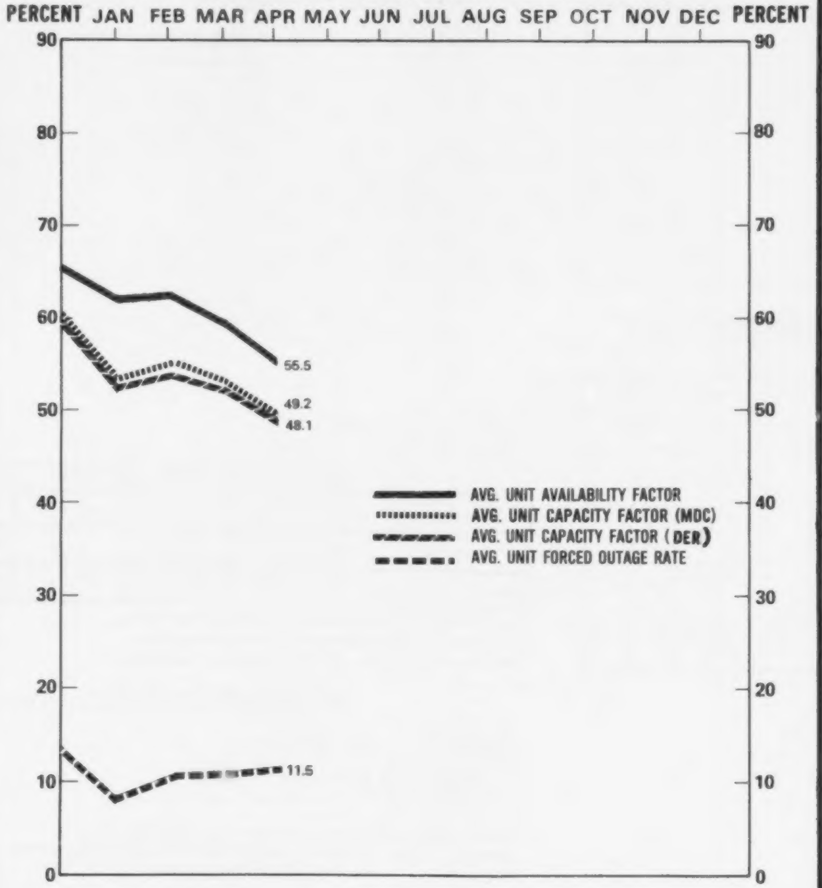
REASONS FOR SHUTDOWN

REASON	NUMBER	TOTAL HOURS LOST
A - Equipment Failure	39	2,457.0
B - Maintenance or Test	14	393.8
C - Refueling	20	11,727.3
D - NRC Restriction	7	4,286.2
E - Operator Training and License Examination	0	0
F - Administrative	1	264,1
G - Operational Error	4	96.6
H - Other	11	2,335.1
TOTAL	96	21,5601

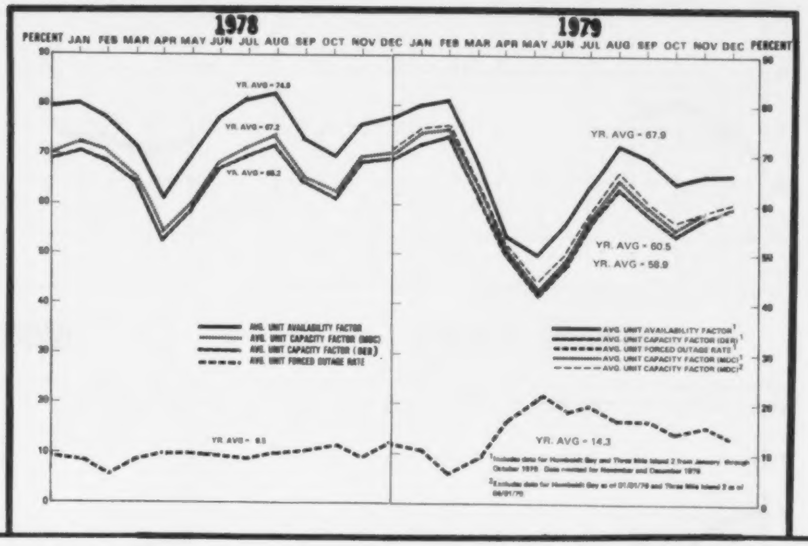
SHUTDOWNS GREATER THAN 72 HOURS EACH

UNIT NAME	REASON	UNIT NAME	REASON
ARKANSAS 1	F	OCONEE 2	C
ARKANSAS 2	H	OCONEE 3	A, D
BEAVER VALLEY 1	D	OYSTER CREEK 1	C
BROWNS FERRY 2	G	PALISADES	D
BRUNSWICK 1	B	PEACH BOTTOM 2	C
BRUNSWICK 2	C	PILGRIM 1	C
COOPER STATION	C	POINT BEACH 1	D
CRYSTAL RIVER 3	C	POINT BEACH 2	C
DAVIS BESSE 1	C	QUAD CITIES 2	A, C
DRESDEN 1	H	RANCHO SECO 1	C
DRESDEN 3	C	ROBINSON 2	A
DUANE ARNOLD	C	SAN ONOFRE 1	C
FORT CALHOUN 1	C	ST LUCIE 1	C
FORT ST VRAIN	A	SURRY 1	D
GINNA	C	SURRY 2	D
HATCH 2	H	THREE MILE ISLAND 1	D
INDIAN POINT 3	A	TROJAN	C
LACROSSE	H	TURKEY POINT 4	C
MONTICELLO	A, C	YANKEE ROWE 1	A
NORTH ANNA 1	H		

UNIT AVAILABILITY, CAPACITY AND FORCED OUTAGE



UNIT AVAILABILITY, CAPACITY AND FORCED OUTAGE



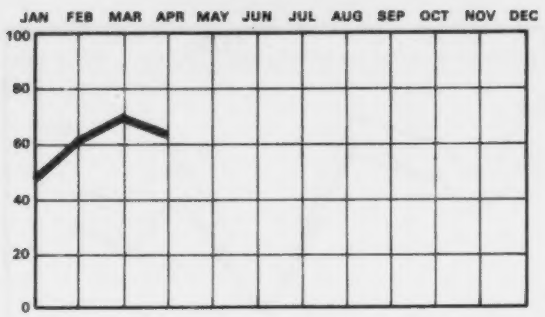
REFUELING SCHEDULE

Please refer to the table on page 3–6 for the scheduled refueling data for each nuclear powerplant in this publication.

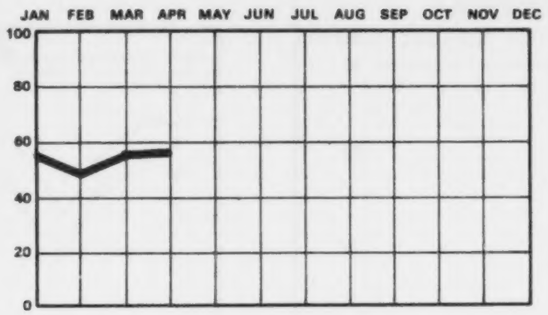
VENDOR AVERAGE CAPACITY FACTORS

APRIL 1980
OPERATIONS

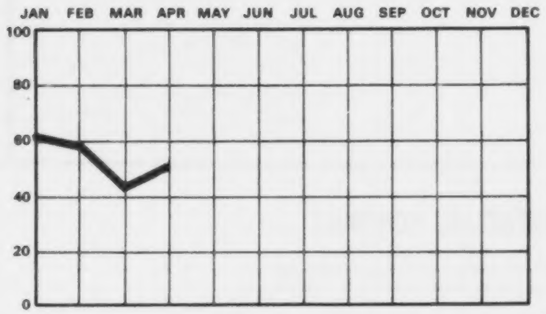
WESTINGHOUSE



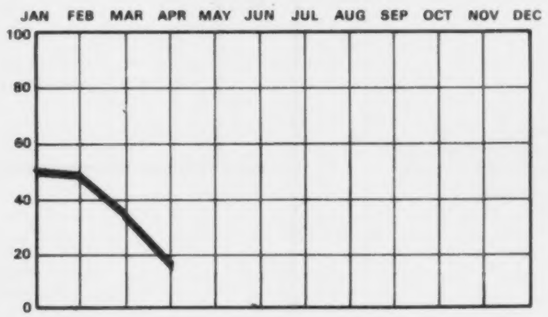
COMBUSTION ENGINEERING



GENERAL ELECTRIC



BABCOCK & WILCOX



NOTE: THIS DISPLAY OF AVERAGE CAPACITY FACTORS PROVIDES A GENERAL PERFORMANCE COMPARISON OF PLANTS SUPPLIED BY THE FOUR NUCLEAR STEAM SUPPLY SYSTEM VENDORS. ONE MUST BE CAREFUL WHEN DRAWING CONCLUSIONS REGARDING THE REASONS FOR THE PERFORMANCE LEVELS INDICATED, SINCE PLANT PERFORMANCE MAY BE AFFECTED BY UNSPECIFIED FACTORS SUCH AS: (1) VARIOUS PLANT DESIGNS AND MODELS ARE INCLUDED FOR EACH VENDOR; (2) TURBINE/GENERATORS AND (3) DIFFERENT ARCHITECT/ENGINEERS ARE ALSO INVOLVED.

GE BWR's	CFMDC
BROWNS FERRY 1	64.0
BROWNS FERRY 2	80.2
BROWNS FERRY 3	100.0
BRUNSWICK 1	38.5
BRUNSWICK 2	.0
COOPER STATION	.0
DRESDEN 1	.0
DRESDEN 2	94.1
DRESDEN 3	.0
DUANE ARNOLD	21.1
FITZPATRICK	80.4
HATCH 1	94.8
HATCH 2	14.0
MILLSTONE 1	98.5
MONTICELLO	45.1
NINE MILE POINT 1	93.1
OYSTER CREEK 1	.0
PEACH BOTTOM 2	.0
PEACH BOTTOM 3	102.3
PILGRIM 1	.0
QUAD CITIES 1	89.4
QUAD CITIES 2	7.1
VERMONT YANKEE 1	96.0

23 UNITS

W PWR's	
BEAVER VALLEY 1	.0
COOK 1	94.7
COOK 2	96.7
FARLEY 1	95.9
GINNA	.0
HADDAM NECK	92.5
INDIAN POINT 2	94.6
INDIAN POINT 3	23.4
KEWAUNEE	98.0
NORTH ANNA 1	78.3
POINT BEACH 1	62.1
POINT BEACH 2	29.1
PRAIRIE ISLAND 1	94.0
PRAIRIE ISLAND 2	91.6
ROBINSON 2	40.7
SALEM 1	96.0
SAN ONOFRE 1	27.4
SURRY 1	.0
SURRY 2	.0
TROJAN	21.2
TURKEY POINT 3	94.9
TURKEY POINT 4	82.4
YANKEE-ROWE 1	.0
ZION 1	91.9
ZION 2	82.2

25 UNITS

CE PWR's	CFMDC
ARKANSAS 2	42.7
CALVERT CLIFFS 1	86.6
CALVERT CLIFFS 2	100.4
FORT CALHOUN 1	0.0
MAINE YANKEE	94.8
MILLSTONE 2	92.3
PALISADES	0.0
ST LUCIE 1	.0

8 UNITS

B&W PWR's	CFMDC
ARKANSAS 1	39.7
CRYSTAL RIVER 3	.0
DAVIS-BESSE 1	14.7
OCONEE 1	87.0
OCONEE 2	.0
OCONEE 3	.0
RANCHO SECO 1	.0
THREE MILE ISLAND 1	.0

8 UNITS

	GE BWR's	W PWR's	CE PWR's	B&W PWR's	All PWR's
NET ELECTRICAL PRODUCTION	6,423,608	8,351,820	2,503,204	862,007	11,717,031
MDC NET	17,327	18,428	6,090	6,737	31,255
CFMDC	51.6	63.1	57.2	17.8	52.2

Capacity factor in this page, denoted as CFMDC, is a function of the net maximum dependable capacity. See the corresponding definition in the Glossary. The vendor averages are computed by the formula :

$$\frac{\text{Net Electrical Energy Produced by Vendor}}{\text{Potential Electrical Production By Vendor In This Month.}} \times 100\%$$

Units Excluded	Reason for Exclusion
Indian Point 1 Humboldt Bay Three Mile Island 2	Unit Shutdown Indefinitely
Big Rock Point	Not Representative of Typical BWRs
LaCroise	Not Representative of Typical BWRs and Atypical Vendor
Fort St. Vrain	Atypical Vendor and Reactor Design

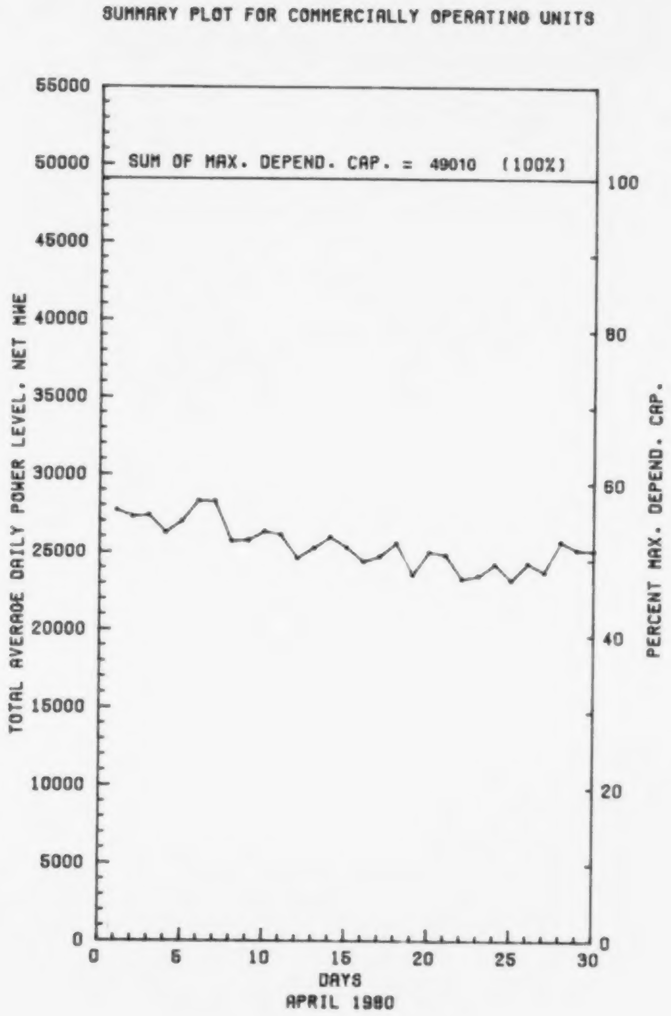
AVERAGE DAILY POWER LEVEL FOR ALL COMMERCIALLY OPERATING UNITS

This chart depicts the average daily power level for the units in commercial operation during the month.

The straight line on the graph labelled "SUM OF MAX. DEPEND. CAP." is plotted at the value shown by summing the separate maximum dependable capacities of the commercially operating units (in Net MWe). The plot shown below the line is calculated by summing the separate average daily power levels of the commercially operating units for each day of the month.

The scale on the left vertical axis runs in 1,000 MWe increments from 0 to 55,000 MWe (Net). The right vertical axis shows the percentage in 10% increments, up to 100% of the "SUM OF MAX. DEPEND. CAP."

It should be recognized that the 100% line would be obtainable only if all of the commercially operating units operated at 100% capacity, 24 hours per day, for the entire month. In other words, since any power generator must occasionally shut down to refuel and/or perform needed maintenance, and also since 100% capacity production is not always required by power demands, the 100% line is a theoretical goal and not a practical one.



ERRATA

Corrections and Amendments to Previously-Reported Data

(NOTE: These changes are reflected in the data contained in the current report)

CORRECTIONS TO NUREG 0020 VOL. 4 NO. 4 APRIL 1980

Page	Unit Name	Item	Correction
	NONE		

MULTIPLE - ISSUE CORRECTIONS

Nine Mile Point 1

Item 16. Gross Thermal Energy (MWH) should be:

Date	Month	Lifetime
Dec 1978		92,331,701
Jan 1979	1,088,921	93,420,622
Feb 1979	832,793	94,253,414
Mar 1979	59,974	94,313,388
Apr 1979	0	0
May 1979	0	0
June 1979	248,321	94,561,709
July 1979	1,230,350	95,792,059
Aug 1979	1,166,861	96,958,920
Sept 1979	1,254,965	98,213,885
Oct 1979	868,282	99,082,167
Nov 1979	1,306,622	100,388,789
Dec 1979	1,352,609	101,741,398
	4,636,727	106,378,125

Rancho Seco

Item 16. Gross Thermal Energy (MWH) should be:

	This Month	Year to Date	Cumulative
Dec 78	574,582	15,929,758	45,067,481
Jan 79	1,929,267	1,929,267	46,996,748
Feb 79	1,638,703	3,567,970	48,635,451
Mar 79	2,033,915	5,601,885	50,669,366
Apr 79	1,723,326	7,325,211	52,392,692
May 79	0	7,325,211	52,392,692
Jun 79	0	7,325,211	52,392,692
Jul 79	1,458,044	8,783,255	53,850,736
Aug 79	1,997,139	10,780,394	55,847,875
Sep 79	1,839,441	12,619,835	57,687,316
Oct 79	1,749,961	14,369,796	59,437,277
Nov 79	1,619,434	15,989,230	61,056,711
Dec 79	1,976,987	17,966,217	63,033,698
Jan 80	689,820	689,820	63,723,518
Feb 80	0	689,820	63,723,518
Mar 80	0	689,820	63,723,518
Apr 80	0	689,820	63,723,518

SECTION 2 – OPERATING POWER REACTORS

ARKANSAS 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Arkansas/Pope 6 miles NW of Russellville
2.	Electric Reliability Council	Southwest Power Pool
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2568
5.	Design Electrical Rating (MWE Net)	850
6.	Date of Initial Criticality	August 6, 1974
7.	Date of Commercial Operation	December 19, 1974
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Dardanelle Reservoir

Utility & Contractor Information

10.	Licensee	Arkansas Power and Light Company
11.	Corporate Address	P.O. Box 551 Little Rock, Arkansas 72203
12.	Corporate Contact	W. Cavanaugh III Vice Pres., Generation & Construction
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	B&W
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	IV
17.	IE Branch Chief	G. L. Madsen
18.	IE Section Chief	T. F. Westerman
19.	IE Resident Inspector	W. D. Johnson (501) 968-3290
20.	Licensing Project Manager	G. S. Vissing
21.	Docket Number	50-313
22.	License Number/Date of Issue	DPR-51, 5/21/74
23.	Public Document Room	Arkansas Polytechnic College Russellville, Arkansas 72801

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO 10 CFR 50.59B, THE LICENSEE FAILED TO RETAIN WRITTEN SAFETY EVALUATIONS FOR TWO DESIGN CHANGES IN THE FACILITY AS DESCRIBED IN THE FSAR.
(8005 DEFICIENCY)

CONTRARY TO TECH. SPEC. 4.4.1.2.5 THE UNIT 1 REACTOR BUILDING PERSONNEL HATCH DOOR WAS NOT LEAK TESTED AFTER PERSONNEL ENTRIES INTO THE REACTOR BUILDING ON 20,21, AND 22 JUNE AND 20 OCT. 1979.
(8005 INFRACTION)

CONTRARY TO A COMMITMENT MADE BY THE LICENSEE IN A LETTER TO THE COMMISSION DATED SEPTEMBER 24, 1979, IN RESPONSE TO I E BULLETIN 79-21, THE LICENSEE FAILED TO PROVIDE POST ACCIDENT LEVEL INSTRUMENT CORRECTION FACTORS TO THE PLANT OPERATORS.
(8005 DEVIATION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NUMBER 2 LOW PRESSURE TURBINE HAS DISKS REMOVED DUE TO STRESS CRACKING DISCOVERED DURING THE LAST OUTAGE. GENERATOR OUTPUT IS REDUCED BY ABOUT 10%.

FACILITY ITEMS (PLANS AND PROCEDURES):

MANAGERIAL ITEMS

PLANT STATUS:

ON 4/19/80, THE UNIT WAS SHUT DOWN FOR MODIFICATIONS TO NON NUCLEAR INSTRUMENTATION SYSTEM. +

LAST IE SITE INSPECTION DATE: FEBRUARY 22 - MARCH 21, 1980 +

INSPECTION REPORT NO: 50-313/80-05 +

SUMMARY

INSPECTION SUMMARY

+ INSPECTION CONDUCTED DURING PERIOD OF FEBRUARY 22 - MARCH 21, 1980 (80-05): ROUTINE, ANNOUNCED INSPECTION INCLUDING SURVEILLANCE TESTING, SURVEILLANCE TEST WITNESSING, FOLLOW-UP ON PREVIOUSLY IDENTIFIED ITEMS, AND FOLLOW-UP ON IE BULLETINS. WITHIN THE FOUR AREAS INSPECTED, TWO ITEMS OF NONCOMPLIANCE AND ONE DEVIATION WERE IDENTIFIED (DEFICIENCY - UNREVIEWED SAFETY QUESTION EVALUATION, INFRACTION - LEAK TESTS OF PERSONNEL AIR LOCK, DEVIATION - IE BULLETIN COMMITMENT).

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

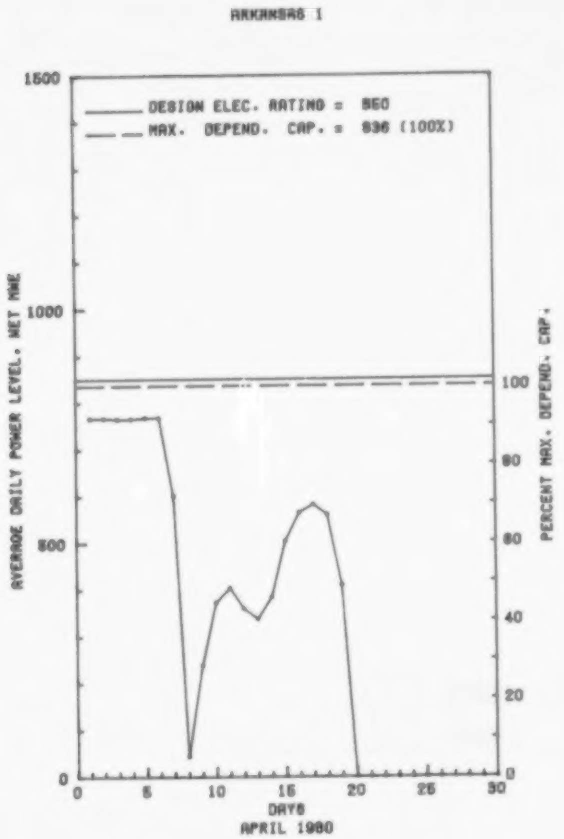
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-05 03L-0	3/6/80	4/7/80	EFW PUMP STEAM SUPPLY VALVE FAILURE TO CLOSE
80-06 03L-0	3/18/80	4/16/80	AVERAGE GROSS GAS RELEASE RATE
80-08 03L-0	3/24/80	4/22/80	PENETRATION ROOM VENTILATION SYSTEM CONTROL VALVE FAILURE
80-10 03L-0	3/7/80	4/8/80	INOPERABLE HANGERS

REVIEWED BY OMPA LOEB: ______ **DATE:** 6/12/80

REVIEWED BY NRR DOR: ______ **DATE:** 6/13/80

ARKANSAS 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-313
2. Utility Contact: L. S. Bramlett (501) 968-2519
3. Licensed Thermal Power (MWt): 2568
4. Nameplate Rating (Gross MWe): 1003 x 0.9 = 903
5. Design Electrical Rating (Net MWe): 850
6. Maximum Dependable Capacity (Gross MWe): 883
7. Maximum Dependable Capacity (Net MWe): 836
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	719.0	2,903.0	47,034.0
12. Hours Reactor Was Critical	437.6	1,581.7	31,719.9
13. Reactor Reserve Shutdown Hours	281.4	374.6	4,622.0
14. Hours Generator On-Line	434.2	1,562.3	31,032.8
15. Unit Reserve Shutdown Hours	20.8	20.8	817.5
16. Gross Thermal Energy (MWH)	826,633	3,642,782	74,904,826
17. Gross Electrical Energy (MWH)	253,838	1,137,998	24,859,154
18. Net Electrical Energy (MWH)	238,918	1,083,274	23,717,310
19. Unit Service Factor	60.4	53.8	66.0
20. Unit Availability Factor	63.3	54.5	67.7
21. Unit Capacity Factor (MDC Net)	39.7	44.6	60.3
22. Unit Capacity Factor (DER Net)	39.1	43.9	59.3
23. Unit Forced Outage Rate	4.6	7.4	15.2

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 05/05/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-02	04/07/80	F	20.7	H	3		ZZ	ZZZZZZ	THE CAUSE OF THE OUTAGE WAS THE LOSS OF ALL OFF SITE POWER BECAUSE OF TORNADO DAMAGE TO TRANSMISSION LINES.
80-03	04/19/80	S	264.1	F	1		ZZ	ZZZZZZ	THE OUTAGE WAS ADMINISTRATIVELY AUTHORIZED FOR ECONOMIC REASONS.

SUMMARY

OPERATION WAS ROUTINE UNTIL 04/07/80, WHEN THE UNIT TRIPPED OFF LINE DUE TO LOSS OF ALL 500 KV LINES DUE TO TORNADO DAMAGE. FOLLOWING RETURN TO POWER ON 04/08/80, LOAD WAS LIMITED BECAUSE OF LINE DAMAGE ACROSS THE STATE. THE UNIT WAS TAKEN OFF LINE FOR THE REST OF THE REPORT PERIOD ON 04/19/80, WITH UNIT 2 CARRYING THE LOAD. MAINTENANCE DURING THE OUTAGE INCLUDED CRYSTAL RIVER MODIFICATIONS, HIGH PRESSURE TURBINE STEAM SEAL REPAIR, AND REPAIR OF A LOW PRESSURE TURBINE/CONDENSER EXPANSION JOINT.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

ARKANSAS 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Arkansas/Pope 6 miles NW of Russellville
2.	Electric Reliability Council	Southwest Power Pool
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2815
5.	Design Electrical Rating (MWE Net)	912
6.	Date of Initial Criticality	12-5-78
7.	Date of Commercial Operation	3/26/80
8.	Condenser Cooling Method	Recirculating
9.	Condenser Cooling Water Source	Cooling Tower

Utility & Contractor Information

10.	Licensee	Arkansas Power & Light Company
11.	Corporate Address	P.O. Box 551 Little Rock, Arkansas 72203
12.	Corporate Contact	W. Cavanaugh III Vice Pres., Generation & Construction
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Combustion Engineering, Inc.
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	IV
17.	IE Branch Chief	G. L. Madsen
18.	IE Section Chief	T. F. Westerman
19.	IE Resident Inspector	W. D. Johnson (501) 968-3290
20.	Licensing Project Manager	R. E. Martin 492-8087
21.	Docket Number	50-368
22.	License Number/Date of Issue	NFP-6, 9/1/78
23.	Public Document Room	Arkansas Polytechnic College Russellville, Arkansas 72801

INSPECTION STATUS

ENFORCEMENT STATUS

THE LICENSEE FAILED TO SET ESF TIME DELAY RELAYS TO MEET THE REQUIREMENTS OF TECHNICAL SPECIFICATION 4.8.1.1.2.C.2.
(8005 INFRACTION)

THE LICENSEE FAILED TO ADHERE TO THE TESTING AND RECORD KEEPING REQUIREMENTS OF SURVEILLANCE PROCEDURE 2304.87, TESTING OF ESF TIME DELAY RELAYS.
(8005 INFRACTION)

CONTRARY TO A COMMITMENT MADE BY THE LICENSEE IN A LETTER TO THE COMMISSION DATED SEPTEMBER 24, 1979, IN RESPONSE TO I E BULLETIN 79-21, THE LICENSEE FAILED TO PROVIDE POST ACCIDENT LEVEL INSTRUMENT CORRECTION FACTORS TO THE PLANT OPERATORS.
(8005 DEVIATION :

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

FACILITY ITEMS (PLANS AND PROCEDURES):

MANAGERIAL ITEMS:

PLANT STATUS:

POWER OPERATION.

LAST IE INSPECTION DATE: FEBRUARY 22 - MARCH 21, 1980

INSPECTION REPORT NO: 50-368/80-05

SUMMARY

INSPECTION SUMMARY

INSPECTION CONDUCTED DURING PERIOD OF FEBRUARY 22 - MARCH 22, 1980 (80-05): ROUTINE, ANNOUNCED INSPECTION INCLUDING SURVEILLANCE TESTING, SURVEILLANCE TEST WITNESSING, INTEGRATED SAFEGUARDS TEST WITNESSING, POWER ASCENSION TEST WITNESSING, FOLLOW-UP ON IE BULLETINS, AND FOLLOW-UP ON PREVIOUSLY IDENTIFIED ITEMS. WITHIN THE SIX AREAS INSPECTED, TWO ITEMS OF NONCOMPLIANCE AND ONE DEVIATION WERE IDENTIFIED (INFRACTION - PROCEDURE ADHERENCE, INFRACTION - SETTING ESF TIME DELAY RELAYS, DEVIATION - IE BULLETIN COMMITMENTS).

REPORTS RECEIVED FROM LICENSEE

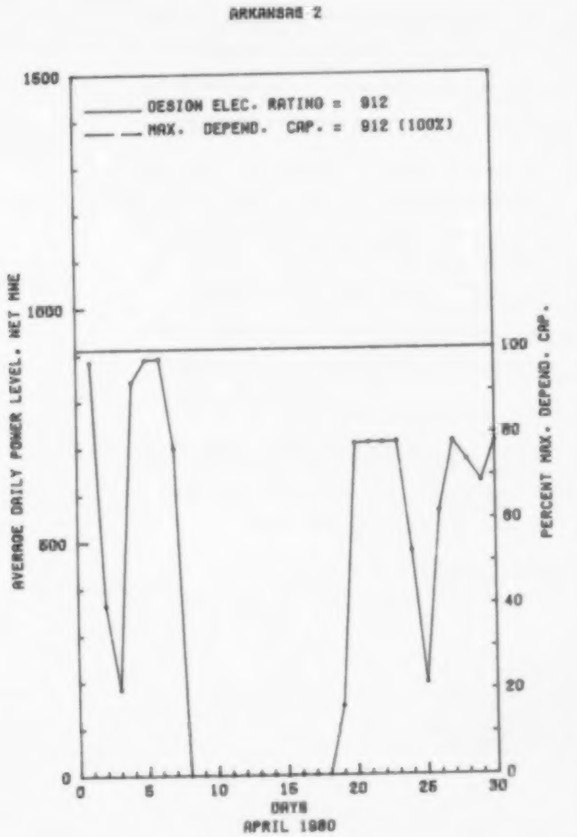
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-13 03L-0	2/29/80	3/28/80	REACTOR COOLANT CHLORIDE CONCENTRATION
80-14 03L-0	3/13/80	4/11/80	CONTAINMENT PENETRATION EXCESSIVE LEAK RATE
80-16 03T-0	3/18/80	4/1/80	SHUTDOWN MARGIN TECHNICAL SPECIFICATION
80-17 03L-0	3/16/80	4/15/80	STEAM GENERATOR PRESSURE INDICATOR FAILURE

REVIEWED BY OMPA LOEB: RE Boba **DATE:** 6/12/80

REVIEWED BY NRR DOR: RE Mart **DATE:** 6/13/80

ARKANSAS 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-368
2. Utility Contact: Rex Pendergraft (501) 968-2519
3. Licensed Thermal Power (MWt): 2815
4. Nameplate Rating (Gross MWe): 959
5. Design Electrical Rating (Net MWe): 912
6. Maximum Dependable Capacity (Gross MWe): 912
7. Maximum Dependable Capacity (Net MWe): 912
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	719.0	863.0	863.0
12. Hours Reactor Was Critical	409.0	533.0	533.0
13. Reactor Reserve Shutdown Hours	69.4	80.9	80.9
14. Hours Generator On-Line	396.2	506.6	506.6
15. Unit Reserve Shutdown Hours	75.0	75.0	75.0
16. Gross Thermal Energy (MWH)	895,638	1,191,867	1,191,867
17. Gross Electrical Energy (MWH)	296,630	392,140	392,140
18. Net Electrical Energy (MWH)	280,016	372,981	372,981
19. Unit Service Factor	54.8	58.7	58.7
20. Unit Availability Factor	65.3	67.4	67.4
21. Unit Capacity Factor (MDC Net)	42.7	47.4	47.4
22. Unit Capacity Factor (DER Net)	42.7	47.4	47.4
23. Unit Forced Outage Rate	45.2	42.1	42.1

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-09	04/02/80	F	24.7	A	3		ZZ	ZZZZZZ	LOST FEED CONTROL DUE TO INADVERTANTLY CYCLING CONTROL BREAKER.
80-10A	04/07/80	F	278.3	H	3		ZZ	ZZZZZZ	LOST OFFSITE POWER (100% LOAD REJECTION) DUE TO WEATHER CONDITIONS. UNIT STAYED OFF LINE DUE TO LACK OF TRANSMISSION CAPABILITY.
80-11	04/24/80	F	17.7	A	3		RB	INSTRU	CEAC ROD POSITION INDICATION FAILURE DUE TO LOSS OF CONTROL ROD POSITION INDICATION.
80-12	04/25/80	F	4.1	A	3		RB	INSTRU	CEAC ROD POSITION INDICATION WAS INTERMITTANTLY LOST. CEAC'S WERE PUT IN THE INOP. MODE FOLLOWING OUTAGE #80-12.

SUMMARY

A REACTOR TRIP OCCURRED ON 04/02/80 FROM LOSS OF A MAIN FEEDWATER PUMP WHEN A BREAKER IN ONE OF THE BREAKER PANELS WAS CYCLED. WEATHER CONDITIONS CAUSED LOSS OF OFFSITE POWER ON 04/07/80, AND THE UNIT REMAINED SHUTDOWN UNTIL 04/19/80 FOR MAINTENANCE. UNIT TRIPS OCCURRED ON 04/24/80 AND 04/25/80 DUE TO INTERMITTENT LOSS OF CONTROL ELEMENT ASSEMBLY POSITION INDICATION.

LEGEND

[1] Type:
- F–Forced
- S–Scheduled

[2] Reason:
- A–Equipment Failure
- B–Maintenance or Test
- C–Refueling
- D–Regulatory Restriction
- E–Operator Training & License Examination
- F–Administrative
- G–Operational Error
- H–Other

[3] Method:
- 1–Manual
- 2–Manual Scram
- 3–Automatic Scram
- 4–Continuations
- 5–Load Reduction
- 9–Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

NRC POWER RESTRICTIONS: NONE

BEAVER VALLEY 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Pennsylvania/Beaver 5 miles E of E. Liverpool, Ohio
2.	Electric Reliability Council	East Central Area Reliability Coordination Agreement
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2660
5.	Design Electrical Rating (MWE Net)	852
6.	Date of Initial Criticality	May 10, 1976
7.	Date of Commercial Operation	10/01/76 (50%) 04/30/77 (100%)
8.	Condenser Cooling Method	Cooling tower/Closed loop
9.	Condenser Cooling Water Source	Ohio River

Utility & Contractor Information

10.	Licensee	Duquesne Light Company
11.	Corporate Address	435 Sixth Avenue Pittsburgh, Pennsylvania 15219
12.	Corporate Contact	C. N. Dunn, Vice President
13.	Architect/Engineer	Stone & Webster
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Section Chief	E C. MCCABE
19.	IE Resident Inspector	D. A. BECKMAN (412-643-9200/9201)
20.	Licensing Project Manager	D. L. Wigginton
21.	Docket Number	50-334
22.	License Number/Date of Issue	DPR-66, 1/30/76
23.	Public Document Room	B. F. Jones Memorial Library 633 Franklin Avenue Aliquippa, PA 15001

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO 10 CFR 50, APP. B, CRITERION XII; THE BVPS FSAR, APPENDIX A.2.2.12; AND, THE BVPS MAINTENANCE MANUAL, CHAPTER 1, SECTION 8, REV. 4, EIGHT INSTRUMENTS ASSOCIATED WITH BOTH EMERGENCY DIESEL GENERATORS AND TWO INSTRUMENTS ASSOCIATED WITH THE MAIN CARDOX FIRE SUPPRESSION UNIT WERE NOT CALIBRATED ON OR BEFORE THEIR REQUIRED DUE DATES. ADDITIONALLY, ONE INSTRUMENT ASSOCIATED WITH THE CARDOX UNIT WAS NOT INCLUDED ON THE APPROVED CALIBRATION SCHEDULE AND WAS NOT THE SUBJECT OF AN APPROVED CALIBRATION PROCEDURE.
(8001 DEFICIENCY)

CONTRARY TO TS 3.3.3.1, ON JANUARY 23, 1980, RADIATION MONITORS RM-VS-104A, RM-VS-104B, AND RM-215B HAD INCORRECTLY ESTABLISHED ALARM/TRIP SETPOINTS IN THAT THE SETPOINTS FOR OPERATION IN MODE 6 (REFUELING) HAD NOT BEEN ESTABLISHED PRIOR TO ENTERING THAT MODE.
(8001 INFRACTION)

CONTRARY TO TS 6.8.1.A; REGULATORY GUIDE 1.33-1972; AND THE BVPS OPERATING MANUAL, CHAPTER 1.48.D.5, REV. 7, A TEMPORARY JUMPER WAS LEFT INSTALLED ON RCS TEMPERATURE RECORDER NO. T-RC-448B WITHOUT IMPLEMENTING THE CONTROLS REQUIRED BY THE ABOVE.
(8001 DEFICIENCY)

CONTRARY TO TS 6.8.1.C AND THE BVPS CHEMISTRY MANUAL, CHAPTER 3, PART 7, REACTOR PLANT SAMPLE SYSTEM, REVISION 1, A PLANT CHEMIST FAILED TO FOLLOW THE APPROVED CHEMISTRY MANUAL PROCEDURES FOR RETURN TO NORMAL VALVE ALIGNMENTS FOLLOWING THE PROCESSING OF A PRIMARY SAMPLE ON FEBRUARY 7, 1980 IN THAT THE COMPLETE PROCEDURE WAS NOT PROPERLY PERFORMED.
(8001 INFRACTION)

10 CFR 50, APP B, CRITERION V STATES IN PART, "ACTIVITIES AFFECTING QUALITY SHALL BE PRESCRIBED BY DOCUMENTED INSTRUCTIONS, PROCEDURES...AND SHALL BE ACCOMPLISHED IN ACCORD WITH THESE INSTRUCTIONS, PROCEDURES..." LICENSEE MAINTENANCE MANUAL CHAPTER 1, CONDUCT OF MAINTENANCE, SECTION J, HOUSEKEEPING, REVISION 4 DTD 9/10/79 STATES IN PART, "THE ENTIRE STATION WILL BE TOURED ONCE A MONTH...RESPONSIBLE SUPERVISORS...ENSURE THAT ALL AREAS ARE CLEANED..." CONTRARY TO THE ABOVE, THE LICENSEE DID NOT CONDUCT STATION HOUSEKEEPING TOURS DURING THE MONTHS OF 12/79 & 1/80.
(8003 INFRACTION)

CONTRARY TO TS 6.8.1.A, RG 1.33, APP A, PARAGRAPH H.1, & STATION QA PROCEDURE OP-12, DATA WAS NOT RECORDED FOR CALIBRATION OF A RAMP GENERATOR USED AS A TEST INSTRUMENT IN THE FUNCTIONAL TEST OF A REACTOR COOLANT LOOP FLOW CHANNEL, & DATA WAS OUT OF TOLERANCE FOR CALIBRATION OF DECADE RESISTANCE BOX USED AS A TEST INSTRUMENT IN CALIBRATION OF A DELTA T-TAVG PROTECTION CHANNEL.
(8005 DEFICIENCY)

CONTRARY TO TS 6.8.1.A, RG 1.33, APP A, PARAGRAPH A & BVPS-OM 1.48.3.E.6 & 1.48.9.L, A LOG SHEET WAS IN USE LOGGING METEORLOGICAL INSTRUMENTS WHICH DID NOT HAVE THE LATEST TEMPORARY CHANGE POSTED, & THE LATEST REVISION WAS NOT POSTED TO COPIES OF THE OPERATING MANUALS IN THE CONTROL ROOM & CONTROL ROOM SHIFT SUPERVISORS OFFICE.
(8005 DEFICIENCY)

CONTRARY TO TS 6.8.1.A & RG 1.33, APP A, PARAGRAPH H.1, A PROCEDURE PREPARED FOR CALIBRATION OF REFUELING WATER STORAGE TANK LEVEL INSTRUMENT LOOP "D" DID NOT CONTAIN THE PROPER CALIBRATION INPUT SIGNALS TO REFLECT A MODIFICATION TO THIS LOOP "D" LEVEL RECORDER WAS ACCOMPLISHED W/O USE OF THE EXISTING PROCEDURE FOR THIS CALIBRATION.
(8005 INFRACTION)

CONTRARY TO TS 6.8.1.A & 6.10.1.D & RG 1.33, APP A, PARAGRAPH H.2, CALIBRATIONS OF SEISMIC & METEOROLOGICAL INSTRUMENTATION AS REQUIRED BY TS WERE PERFORMED BY USE OF VENDOR PROCEDURES WHICH WERE UNREVIEWED & UNAPPROVED BY THE LICENSEE & RECORDS OF METEOROLOGICAL CALIBRATIONS WERE NOT MAINTAINED BY THE LICENSEE.
(8005 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

FACILITY ITEMS (PLANS AND PROCEDURES):

Current outage expected to last until 7/80.

PLANT STATUS:

Shutdown since 11/79 for refueling, modification, and maintenance.

LAST IE SITE INSPECTION DATE: 4/24/80

INSPECTION REPORT NO: 50-334/80-09

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-001/ 5P	02/26/80	03/05/80	Shut down motor driven fire pump 00S due overheated bearing. Portable diesel pump to be used for water supply for outage activities
80-002/ 5P	02/25/80	04/03/80	False alarms due to the malfunctions of seismic instrumentation
80-007/ 01T	02/21/80	03/04/80	Supplementary leak flow not lined up to main filter as required during portion of refueling operation
80-008/ 01T	02/25/80	03/04/80	Potential SSE overstress of cable spreading room floor beams from horizontal acceleration of conduit and tray supports on lower flange of the steel beams
80-009/ 03L	02/01/80	02/29/80	Emergency diesel load sequencer inop due to improper wiring connections
80-010/ 03L	01/31/80	03/04/80	Refueling core alterations stopped when boric acid transfer pump tripped on thermal overload
80-011/ 03L	02/26/80	03/04/80	Charging system fill to RCS MOV-SI-869A linestarter failure during surveillance, with MOV-CH-1-15C also not closing due to dirty torque switch and MOV-CH-115E used to effect isol
80-012/ 01T	03/12/80	03/31/80	Remote shutdown panel channel checks for aux feed water not done monthly since 8/76 due normal op RDG of zero but TS not changed & alternative testing not instituted
80-013/ 03T	03/02/80	03/20/80	Cooling tower blowdown temp meter 00S high but recalibrated before OPS with blowdown temp above spec (75 degrees F)
80-014/ 04T	03/07/80	03/21/80	Hi-Hi radiation alarm and discharge VLV closure inop due to loose loose relay/solder joints. Recheck of discharges verified no limits exceeded
80-015/ 04T	03/08/80	03/21/80	Component cooling water hi-hi radiation alarm inop due to out of adjustment potentiometer
80-016/ 03L	03/11/80	03/31/80	Remote shutdown panel aux feed flow meters not tested as meter was deleted from data sheet. No affect on public health and safety in future tests. These matters to be checked
80-017/ 01T	03/26/80	04/08/80	Combined main steam safety blowdown, obe, pressure, and dead lead stresses on safety valve header
80-018/ 01T	03/26/80	04/08/80	System pressure failure of fire suppression water system
80-019/ 03L	03/10/80	04/08/80	Stress corrosion crack in a channel head of the liquid waste evaporator bottoms heat exchanger

SUMMARY

INSPECTION SUMMARY

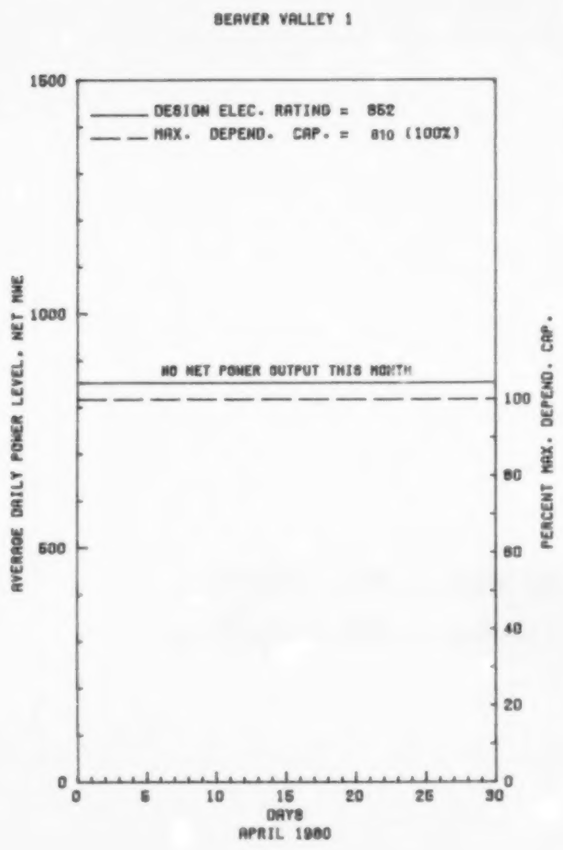
50-334/80-01 - Jan 14 - Feb 8: Routine inspection by the (129 hours). resident inspector and region-based inspectors findings; Inspection areas (8): action on previous inspection operations; plant operations; refueling preparations; refueling of periodic IE Bulletin followup; licensee event review; review security. reports; and review of physical protection/plant administrative Noncompliances (4): Inadequate implementation of setpoints controls for jumpers/lifted leads; Radiation Monitor Chem- set in excess of allowable TS limits; Failure to follow as istry Manual Procedures; Failure to calibrate instruments required.

REVIEWED BY OMPA LOEB: T. C. Cintula **DATE:** 6·12·80

REVIEWED BY NRR DOR: W. J. Ross **DATE:** 6/13/80

BEAVER VALLEY 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-334
2. Utility Contact: DAVID R. TIMKO (412) 643-5308
3. Licensed Thermal Power (MWt): 2660
4. Nameplate Rating (Gross MWe): 1026 x 0.9 = 923
5. Design Electrical Rating (Net MWe): 852
6. Maximum Dependable Capacity (Gross MWe): 845
7. Maximum Dependable Capacity (Net MWe): 810
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	719.0	2,903.0	35,063.0
12. Hours Reactor Was Critical	.0	.0	13,774.7
13. Reactor Reserve Shutdown Hours	.0	.0	4,482.7
14. Hours Generator On-Line	.0	.0	13,105.0
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	0	26,974,253
17. Gross Electrical Energy (MWH)	0	0	8,277,940
18. Net Electrical Energy (MWH)	0	0	7,570,743
19. Unit Service Factor	.0	.0	38.6
20. Unit Availability Factor	.0	.0	38.6
21. Unit Capacity Factor (MDC Net)	.0	.0	31.0
22. Unit Capacity Factor (DER Net)	.0	.0	28.5
23. Unit Forced Outage Rate	.0	.0	48.4

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 07/22/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
1	12/01/79	S	719.0	D	4		ZZ	ZZZZZZ	UNIT SHUTDOWN FOR MAJOR MODIFICATIONS AS REQUIRED BY THE NUCLEAR REGULATORY COMMISSION, INCLUDING NRC BULLETINS IEB 79-02 AND 79-14.

SUMMARY

THE UNIT REMAINED SHUTDOWN THROUGHOUT THE REPORT PERIOD FOR MAJOR NRC-REQUIRED MODIFICATIONS.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

Exhibit H:
Same source.

BIG ROCK POINT 1

FACILITY DATA

Facility Description

1. Location (state/county) (nearest population center)	Michigan/Charlevoix 4 miles NE of Charlevoix
2. Electric Reliability Council	East Central Area Reliability Coordination Agreement
3. Type of Reactor	BWR
4. Power Level (MWT)	240
5. Design Electrical Rating (MWE Net)	72
6. Date of Initial Criticality	September 27, 1962
7. Date of Commercial Operation	March 29, 1963
8. Condenser Cooling Method	Once Through
9. Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10. Licensee	Consumers Power Company
11. Corporate Address	212 West Michigan Avenue Jackson, Michigan 49201
12. Corporate Contact	R. B. DeWitt
13. Architect/Engineer	Bechtel Corporation
14. Nuclear Steam Supply System	GE
15. Constructor	Bechtel Corporation

Regulatory Information

16. IE Region responsible	III
17. IE Branch Chief	R. Heishman
18. IE Section Chief	K. R. BAKER
19. IE Resident Inspector	G. Wright
20. Licensing Project Manager	W. A. Paulson
21. Docket Number	50-155
22. License Number/Date of Issue	DPR-6 issued 8/30/62
23. Public Document Room	Charlevoix Public Library 107 Clinton Street Charlevoix, Michigan

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

Routine operation.

*LAST IE SITE INSPECTION DATE: February 28-29 and March 10-14, 1980

* INSPECTION REPORT NO: 80-03

SUMMARY

INSPECTION SUMMARY

* Inspection on February 28-29 and March 10-14, (80-03): Routine, unannounced inspection of the licensee's training and retraining program; surveillance activities; calibration program; procedure changes and training with regard to Small Break Loss of Coolant Accident (SBLOCA); and followup actions relative to IE Bulletins, IE Circulars, and Licensee Event Reports. The inspection involved 65 inspector hours onsite by two NRC inspectors. No items of noncompliance or deviations were identified.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

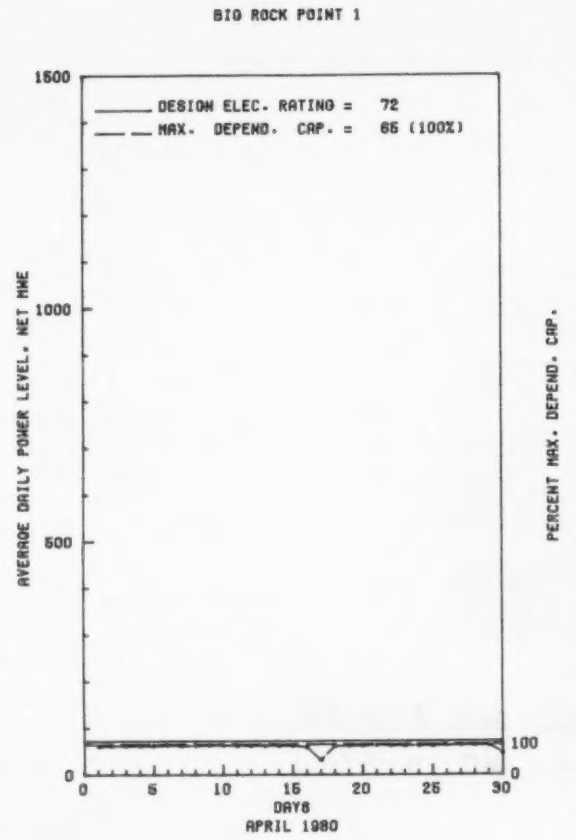
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-05	03/01/80	04/02/80	Following several intermittent automatic test circuit alarms, the sensor channel D of the Reactor Depressurization System was removed from service to permit troubleshooting.

REVIEWED BY OMPA LOEB: DATE: 6/13/80

REVIEWED BY NRR DOR: W. A. Paulson DATE: 6/16/80

BIG ROCK POINT 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-155
2. Utility Contact: G. Himebauch (616) 547-6537 x180
3. Licensed Thermal Power (MWt): 240
4. Nameplate Rating (Gross MWe): 70.6 x 0.85 = 60
5. Design Electrical Rating (Net MWe): 72
6. Maximum Dependable Capacity (Gross MWe): 67
7. Maximum Dependable Capacity (Net MWe): 65
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): 63
10. Reasons for Restrictions, If Any: FUEL THERMAL HYDRAULIC LIMITS

ITEM 21 COMPUTED WITH A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	719.0	2,903.0	149,826.0
12. Hours Reactor Was Critical	719.0	2,581.2	102,714.2
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	719.0	2,568.5	100,567.4
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	151,590	541,045	10,650,976
17. Gross Electrical Energy (MWH)	46,104	164,793	5,900,711
18. Net Electrical Energy (MWH)	43,588	155,683	5,584,020
19. Unit Service Factor	100.0	88.5	67.1
20. Unit Availability Factor	100.0	88.5	67.1
*21. Unit Capacity Factor (MDC Net)	93.3	82.5	55.0
22. Unit Capacity Factor (DER Net)	84.2	74.5	51.8
23. Unit Forced Outage Rate	.0	1.5	24.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUELING - OCTOBER 1980 - 6 TO 8 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
4	04/17/80	S	26.5	H	5		HJ	ZZZZZZ	POWER REDUCTION TO FACILITATE REPAIRS IN HIGH PRESSURE TURBINE DRAIN LINE.
5	04/30/80	S	9.0	H	5		HE	ZZZZZZ	POWER REDUCTION TO FACILITATE PIPING REPAIR DOWNSTREAM OF TURBINE BYPASS VALVE.

SUMMARY

OPERATION WAS ROUTINE AT A POWER RESTRICTION TO 63 MWE (NET) DUE TO FUEL THERMAL HYDRAULIC LIMITS. LOAD WAS FURTHER REDUCED ON 04/17/80 TO REPAIR THE HIGH PRESSURE TURBINE DRAIN LINE, AND ON 04/30/80 TO REPAIR PIPING DOWNSTREAM OF A TURBINE BYPASS VALVE.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

BROWNS FERRY 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Alabama/Limestone 10 miles NW of Decatur
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	3293
5.	Design Electrical Rating (MWE Net)	1065
6.	Date of Initial Criticality	August 17, 1973
7.	Date of Commercial Operation	August 1, 1974
8.	Condenser Cooling Method	Once Through With Forced Draft Cooling Towers
9.	Condenser Cooling Water Source	Tennessee River

Utility & Contractor Information

10.	Licensee	Tennessee Valley Authority
11.	Corporate Address	500A Chestnut Street Tower II Chattanooga, Tennessee 37401
12.	Corporate Contact	H. G. Parris, Manager of Power.
13.	Architect/Engineer	TVA
14.	Nuclear Steam Supply System	GE
15.	Constructor	TVA

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. Lewis
18.	IE Section Chief	H. C. Dance
19.	IE Resident Inspector	R. Sullivan (205) 729-6196
20.	Licensing Project Manager	R. Clark
21.	Docket Number	50-259
22.	License Number/Date of Issue	DPR-33, 6/26/73
23.	Public Document Room	Athens Public Library South and Forrest Athens, Alabama 35611

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

ROUTINE OPERATIONS.

LAST IE SITE INSPECTION DATE: MARCH 7 - APRIL 4, 1980

INSPECTION REPORT NO: 50-259/80-19

SUMMARY

INSPECTION SUMMARY

+ INSPECTION JANUARY 14 - FEBRUARY 1 (80-06): THIS ROUTINE INSPECTION INVOLVED 28 RESIDENT INSPECTOR-HOURS IN THE AREAS OF PLANT OPERATIONS, PLANT TOURS, REPORTABLE OCCURRENCES, REFUELING, NONCOMPLIANCE FOLLOWUP, IE BULLETIN FOLLOWUP, PLANT PHYSICAL PROTECTION AND RADIATION AREA CONTROLS. OF THE EIGHT AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION FEBRUARY 26-28 (80-09): THIS SPECIAL, ANNOUNCED INSPECTION INVOLVED 13 INSPECTOR-HOURS ON SITE IN THE AREAS OF SMALL BREAK LOSS OF COOLANT ACCIDENT PROCEDURES AND TRAINING. OF THE TWO AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION MARCH 11-14 (80-12): INCLUDED REVIEW OF THE LICENSEE'S SECURITY PLAN, SECURITY ORGANIZATION, ACCESS CONTROLS TO PROTECTED AND VITAL AREAS, PROTECTED AND VITAL AREA BARRIERS, AND LICENSEE ACTION ON PREVIOUS INSPECTION FINDINGS. THIS SPECIAL INSPECTION WAS IN RESPONSE TO A SERIES OF REPORTED INCIDENTS AT THE BROWNS FERRY NUCLEAR PLANT, BEGINNING IN EARLY FEBRUARY 1980. THE INSPECTION INVOLVED 10 INSPECTOR-HOURS ON SITE. THE INSPECTION WAS BEGUN DURING A REGULAR PERIOD; 3 INSPECTION HOURS WERE ACCOMPLISHED DURING OFFSHIFT PERIODS. THE LICENSEE WAS FOUND TO BE IN COMPLIANCE WITH NRC REQUIREMENTS WITHIN THE AREAS EXAMINED DURING THE INSPECTION.

INSPECTION FEBRUARY 4 - MARCH 7 (80-13): THIS ROUTINE INSPECTION INVOLVED 46 RESIDENT INSPECTOR-HOURS IN THE AREAS OF PLANT OPERATIONS, PLANT TOURS, REPORTABLE OCCURRENCES, REFUELING, PLANT PHYSICAL PROTECTION, AND RADIATION AREA CONTROLS. OF THE SIX AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN FIVE AREAS. ONE ITEM OF APPARENT NONCOMPLIANCE WAS FOUND IN ONE AREA (INFRACTION - FAILURE TO DEMONSTRATE HPCI OPERABLE WHEN RCIC WAS INOPERATIVE FOR MAINTENANCE).

INSPECTION MARCH 13-14 (80-14): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 12 INSPECTOR-HOURS ON SITE IN THE AREA OF HPCI SYSTEM PIPING AND RESTRAINT WORK ACTIVITIES. OF THE AREA INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION MARCH 19-20 (80-15): THIS SPECIAL ANNOUNCED INSPECTION INVOLVED 4 INSPECTOR-HOURS IN THE AREA OF SEISMIC ANALYSIS FOR AS-BUILT SAFETY-RELATED PIPING SYSTEMS (IE BULLETIN 79-14). NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION MARCH 24-28 AND MARCH 31 - APRIL 4 (80-16): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 28 INSPECTOR-HOURS ON SITE AND 6 INSPECTOR-HOURS OFF SITE IN THE AREAS OF COORDINATION WITH OUTSIDE SUPPORT AGENCIES; EMERGENCY FACILITIES, EQUIPMENT AND PROCEDURES; EMERGENCY TRAINING; EMERGENCY PLANNING AUDITS; EMERGENCY DRILLS; FOLLOWUP ON PREVIOUSLY IDENTIFIED INSPECTION FINDINGS AND FOLLOWUP ON IE BULLETINS. OF THE SEVEN AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN SIX AREAS; TWO ITEMS OF NONCOMPLIANCE WERE FOUND IN ONE AREA (INFRACTION - FAILURE TO PREPARE AND ADHERE TO WRITTEN PROCEDURES; AND INFRACTION - FAILURE TO CONDUCT EMERGENCY TRAINING FOR ALL DESIGNATED EMERGENCY DIRECTORS).

INSPECTION MARCH 7 - APRIL 4 (80-19): THIS ROUTINE INSPECTION INVOLVED 42 RESIDENT INSPECTOR-HOURS IN THE AREAS OF OPERATIONAL SAFETY, MAINTENANCE, REPORTABLE OCCURRENCES, FOLLOWUP ON NONCOMPLIANCE AND UNRESOLVED ITEMS, RADIATION CONTROL, PLANT PHYSICAL PROTECTION AND REACTOR TRIPS. OF THE SEVEN AREAS INSPECTED NO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED IN SIX AREAS. ONE ITEM OF APPARENT NONCOMPLIANCE WAS FOUND IN ONE AREA (INFRACTION - FAILURE TO FOLLOW PROCEDURES IN HEAT-TREATMENT OF UNIT HPCI 1 STEAM PIPING).

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

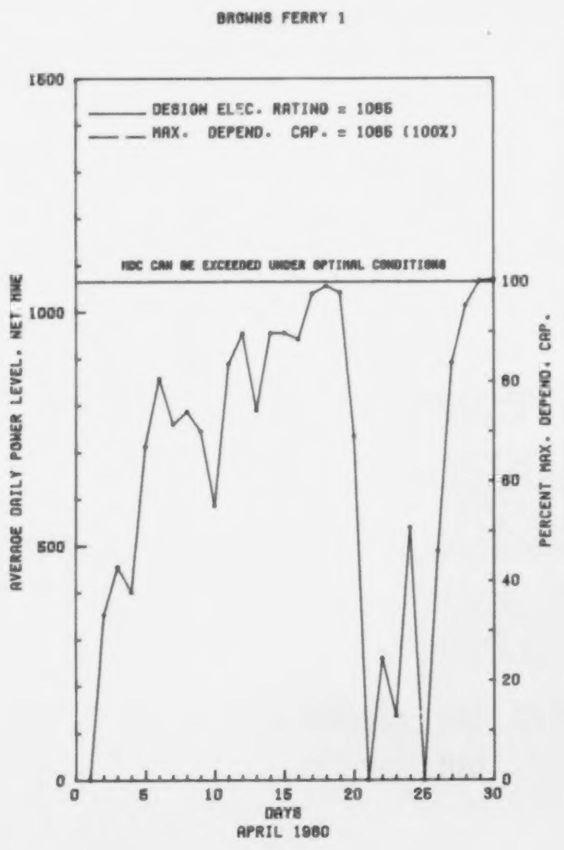
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
80-008/ 03L-0	01/27/80	02/25/80	30 DAY LER	HIGH PRESSURE COOLANT INJECTION PUMP PEDESTAL CRACKED.
80-018/ 03L-0	02/17/80	03/14/80	30 DAY LER	4KV BOARD BRK FAILED TO CLOSE DURING TRANSFER TEST.
80-019/ 01T-0	03/01/80	03/14/80	14 DAY LER	FAILURE OF OFFSITE POWER SUPPLY.
80-022/ 03L-0	02/12/80	03/12/80	30 DAY LER	PRIMARY CONTAINMENT PENETRATIONS EXCESSIVE LEAKAGE.

REVIEWED BY OMPA LOEB: Earl Hall **DATE:** 8-12-80

REVIEWED BY NRR DOR: Dick Clark **DATE:** 06-13-80

BROWNS FERRY 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-259
2. Utility Contact: Don Green (205) 729-6846
3. Licensed Thermal Power (MWt): 3293
4. Nameplate Rating (Gross MWe): 1280 x 0.9 = 1152
5. Design Electrical Rating (Net MWe): 1065
6. Maximum Dependable Capacity (Gross MWe): 1098
7. Maximum Dependable Capacity (Net MWe): 1065
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:

NONE

9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:

NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	719.0	2,903.0	50,401.0
12. Hours Reactor Was Critical	665.7	823.6	29,100.9
13. Reactor Reserve Shutdown Hours	73.3	247.9	4,937.4
14. Hours Generator On-Line	614.1	727.3	28,340.2
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,535,026	1,741,532	75,878,025
17. Gross Electrical Energy (MWH)	502,970	572,960	25,051,420
18. Net Electrical Energy (MWH)	490,352	549,605	24,351,573
19. Unit Service Factor	85.4	25.1	56.2
20. Unit Availability Factor	85.4	25.1	56.2
21. Unit Capacity Factor (MDC Net)	64.0	17.8	45.4
22. Unit Capacity Factor (DER Net)	64.0	17.8	45.4
23. Unit Forced Outage Rate	14.6	28.3	32.9

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

NONE

25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
111	03/29/80	F	22.9	A	4				HIGH TEMPERATURE ON "B" RECIRC. PUMP THRUST BEARING.
112	04/07/80	S	.0	B	5				DERATED FOR RTI 23A (FEEDWATER CONTROL).
113	04/08/80	S	.0	B	5				DERATED FOR RTI 23A (FEEDWATER CONTROL).
114	04/09/80	S	.0	B	5				DERATED FOR RTI 32A (RECIRC. PUMP CONTROLS).
115	04/13/80	S	.0	H	5				DERATED FOR CONTROL ROD ADJUSTMENT.
116	04/15/80	F	.0	A	5				"B" RECIRC. PUMP TRIP.
117	04/20/80	F	36.5	A	2				MAINTENANCE TO [illegible]V-69-1.
118	04/23/80	F	15.0	B	2				MAIN TURBINE BALANCING (HIGH VIBRATION).
119	04/24/80	F	.0	A	5				DERATED DUE TO "B" RECIRC. PUMP LOW OIL LEVEL SIGNAL.
120	04/24/80	F	30.5	A	2				MAINTENANCE TO "B" RECIRC. PUMP (LOW OIL LEVEL SIGNAL) (LOOSE WIRES IN CONDUIT).
121	04/27/80	S	.0	H	5				DERATED FOR CONTROL ROD PATTERN ADJUSTMENT.

SUMMARY

THE UNIT WAS OFF LINE ON 04/01/80 DUE TO HIGH TEMPERATURE ON THE "B" RECIRCULATION PUMP THRUST BEARING. SEVERAL LOAD REDUCTIONS AND TWO MORE OUTAGES WERE NECESSARY FOR MAINTENANCE AND CONTROL ROD ADJUSTMENT.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

BROWNS FERRY 2

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Alabama/Limestone 10 miles NW of Decatur
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	3293
5.	Design Electrical Rating (MWE Net)	1065
6.	Date of Initial Criticality	July 20, 1974
7.	Date of Commercial Operation	March 1, 1975
8.	Condenser Cooling Method	Once Through With Forced Draft Cooling Towers
9.	Condenser Cooling Water Source	Tennessee River

Utility & Contractor Information

10.	Licensee	Tennessee Valley Authority
11.	Corporate Address	500A Chestnut Street Tower II Chattanooga, Tennessee 37401
12.	Corporate Contact	H. G. Parris, Manager of Power
13.	Architect/Engineer	TVA
14.	Nuclear Steam Supply System	GE
15.	Constructor	TVA

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. Lewis
18.	IE Section Chief	H. C. Dance
19.	IE Resident Inspector	R. Sullivan (205) 729-6196
20.	Licensing Project Manager	R. Clark
21.	Docket Number	50-260
22.	License Number/Date of Issue	DPR-52, 6/28/74
23.	Public Document Room	Athens Public Library South and Forrest Athens, Alabama 35611

INSPECTION STATUS

ENFORCEMENT STATUS

AS REQUIRED BY TECHNICAL SPECIFICATION 6.3.A, DETAILED WRITTEN PROCEDURES SHALL BE PREPARED, APPROVED AND ADHERED TO FOR SURVEILLANCE AND TESTING REQUIREMENTS. CONTRARY TO THE ABOVE, ON DECEMBER 21, 1979 A LEAK TEST OF THE UNIT 2 SOUTHEAST EQUIPMENT HATCH WAS PERFORMED USING AN OUTDATED SECTION, 5.1.6, OF SURVEILLANCE INSTRUCTION 4.7.A.2.G-2, PRIMARY CONTAINMENT TESTABLE PENETRATIONS, WHICH HAD BEEN SUPERCEDED BY AN APPROVED REVISION DATED 11/27/79.
(7947 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

100 PERCENT FULL POWER OPERATION.

LAST IE SITE INSPECTION DATE: MARCH 7 - APRIL 4, 1980

INSPECTION REPORT NO: 50-260/80-19

SUMMARY

INSPECTION SUMMARY

INSPECTION JANUARY 14 - FEBRUARY 1 (80-06): THIS ROUTINE INSPECTION INVOLVED 27 RESIDENT INSPECTOR-HOURS IN THE AREAS OF PLANT OPERATIONS, PLANT TOURS, REPORTABLE OCCURRENCES, REFUELING, NONCOMPLIANCE FOLLOWUP, IE BULLETIN FOLLOWUP, PLANT PHYSICAL PROTECTION AND RADIATION AREA CONTROLS. OF THE EIGHT AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION FEBRUARY 26-28 (80-09): THIS SPECIAL, ANNOUNCED INSPECTION INVOLVED 13 INSPECTOR-HOURS ON SITE IN THE AREAS OF SMALL BREAK LOSS OF COOLANT ACCIDENT PROCEDURES AND TRAINING. OF THE TWO AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION MARCH 11-14 (80-10): INCLUDED REVIEW OF THE LICENSEE'S SECURITY PLAN, SECURITY ORGANIZATION, ACCESS CONTROLS TO PROTECTED AND VITAL AREAS, PROTECTED AND VITAL AREA BARRIERS, AND LICENSEE ACTION ON PREVIOUS INSPECTION FINDINGS. THIS SPECIAL INSPECTION WAS IN RESPONSE TO A SERIES OF REPORTED INCIDENTS AT THE BROWNS FERRY NUCLEAR PLANT, BEGINNING IN EARLY FEBRUARY 1980. THE INSPECTION INVOLVED 10 INSPECTOR-HOURS ON SITE. THE INSPECTION WAS BEGUN DURING A REGULAR PERIOD; 2 INSPECTION HOURS WERE ACCOMPLISHED DURING OFFSHIFT PERIODS. THE LICENSEE WAS FOUND TO BE IN COMPLIANCE WITH NRC REQUIREMENTS WITHIN THE AREAS EXAMINED DURING THE INSPECTION.

INSPECTION FEBRUARY 4 - MARCH 7 (80-11): THIS ROUTINE INSPECTION INVOLVED 46 RESIDENT INSPECTOR-HOURS IN THE AREAS OF PLANT OPERATIONS, PLANT TOURS, REPORTABLE OCCURRENCES, REFUELING, PLANT PHYSICAL PROTECTION, AND RADIATION AREA CONTROLS. OF THE SIX AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN FIVE AREAS. ONE ITEM OF APPARENT NONCOMPLIANCE WAS FOUND IN ONE AREA (INFRACTION - FAILURE TO DEMONSTRATE HPCI OPERABLE WHEN RCIC WAS INOPERATIVE FOR MAINTENANCE).

INSPECTION MARCH 19-20 (80-12): THIS SPECIAL ANNOUNCED INSPECTION INVOLVED 4 INSPECTOR-HOURS IN THE AREA OF SEISMIC ANALYSIS FOR AS-BUILT SAFETY-RELATED PIPING SYSTEMS (IE BULLETIN 79-14). NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION MARCH 24-28 AND MARCH 31 - APRIL 4 (80-13): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 20 INSPECTOR-HOURS ON SITE AND 6 INSPECTOR-HOURS OFF SITE IN THE AREAS OF COORDINATION WITH OUTSIDE SUPPORT AGENCIES; EMERGENCY FACILITIES, EQUIPMENT AND PROCEDURES; EMERGENCY TRAINING; EMERGENCY PLANNING AUDITS; EMERGENCY DRILLS; FOLLOWUP ON PREVIOUSLY IDENTIFIED INSPECTION FINDINGS AND FOLLOWUP ON IE BULLETINS. OF THE SEVEN AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN SIX AREAS; TWO ITEMS OF NONCOMPLIANCE WERE FOUND IN ONE AREA (INFRACTION - FAILURE TO PREPARE AND ADHERE TO WRITTEN PROCEDURES; AND INFRACTION - FAILURE TO CONDUCT EMERGENCY TRAINING FOR ALL DESIGNATED EMERGENCY DIRECTORS).

INSPECTION MARCH 7 - APRIL 4 (80-14): THIS ROUTINE INSPECTION INVOLVED 42 RESIDENT INSPECTOR-HOURS IN THE AREAS OF OPERATIONAL SAFETY, MAINTENANCE, REPORTABLE OCCURRENCES, FOLLOWUP ON NONCOMPLIANCE AND UNRESOLVED ITEMS, RADIATION CONTROL, PLANT PHYSICAL PROTECTION AND REACTOR TRIPS. OF THE SEVEN AREAS INSPECTED NO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED IN SIX AREAS. ONE ITEM OF APPARENT NONCOMPLIANCE WAS FOUND IN ONE AREA (INFRACTION - FAILURE TO FOLLOW PROCEDURES IN HEAT-TREATMENT OF UNIT 1 STEAM PIPING).

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

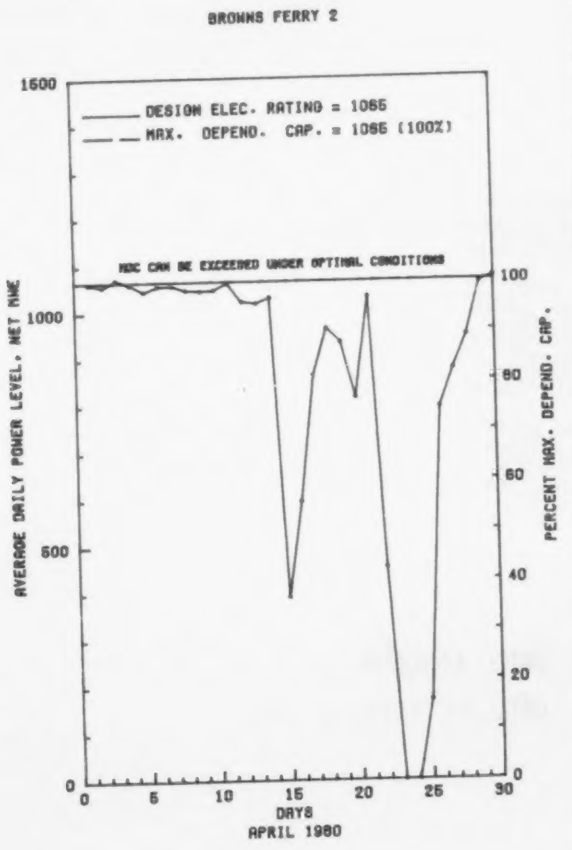
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
80-10/ 03L-0	02/16/80	.03/13/80	30 DAY LER	HIGH PRESSURE COOLANT INJECTION TURBINE PEDESTAL CRACKS.
80-011/ 03L-0	02/17/80	03/17/80	30 DAY LER	480V BOARD FAILED TO PROPERLY TRANSFER DURING TEST.
80-012/ 03L-0	02/23/80	03/19/80	30 DAY LER	HIGH PRESSURE COOLANT INJECTION TURBINE PEDESTAL CRACKS.

REVIEWED BY OMPA LOEB: Earl Hill **DATE:** 6/10/80

REVIEWED BY NRR DOR: Dick Clark **DATE:** 06-13-80

BROWNS FERRY 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-260
2. Utility Contact: Don Green (205) 729-6846
3. Licensed Thermal Power (MWt): 3293
4. Nameplate Rating (Gross MWe): 1280 x 0.9 = 1152
5. Design Electrical Rating (Net MWe): 1065
6. Maximum Dependable Capacity (Gross MWe): 1098
7. Maximum Dependable Capacity (Net MWe): 1065
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	719.0	2,903.0	45,312.0
12. Hours Reactor Was Critical	648.3	2,431.1	26,915.5
13. Reactor Reserve Shutdown Hours	70.8	471.9	12,246.8
14. Hours Generator On-Line	636.0	2,367.5	26,032.0
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,888,675	6,660,849	72,815,849
17. Gross Electrical Energy (MWH)	630,280	2,224,880	24,174,948
18. Net Electrical Energy (MWH)	614,015	2,163,956	23,495,425
19. Unit Service Factor	88.5	81.6	57.5
20. Unit Availability Factor	88.5	81.6	57.5
21. Unit Capacity Factor (MDC Net)	80.2	70.0	48.7
22. Unit Capacity Factor (DER Net)	80.2	70.0	48.7
23. Unit Forced Outage Rate	11.5	18.4	35.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
REFUEL OUTAGE SEPT. 1980.
25. If Shut Down At End of Report Period, Estimated Date or Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
132	04/15/80	F	8.0	G	3				LOW RX WATER LEVEL SIGNAL DUE TO INADVERTANT OPENING OF MAIN STEAM FLOW CONTROL CIRCUIT.
133	04/19/80	F	.0	B	5				DERATED TO TAKE "C" HP STRING HEATERS OUT OF SERVICE FOR TESTING.
134	04/22/80	F	75.0	G	3				HI RX WATER LEVEL SIGNAL DUE TO PERSONNEL ERROR DURING S.I.
135	04/27/80	F	.0	B	5				DERATED TO TAKE "C" STRING L. P. HEATERS OUT OF SERVICE FOR MAINTENANCE.

SUMMARY

OPERATION WAS AT NEAR FULL POWER UNTIL 04/15/80 WHEN THE INADVERTENT OPENING OF THE MAIN STEAM FLOW CONTROL CIRCUIT CAUSED LOW REACTOR WATER LEVEL. THE UNIT WAS TAKEN OFF LINE ON 04/22/80 WHEN HIGH REACTOR WATER LEVEL RESULTED FROM PERSONNEL ERROR DURING A SAFETY INJECTION.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

BROWNS FERRY 3

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Alabama/Limestone 10 miles NW of Decatur
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	3293
5.	Design Electrical Rating (MWE Net)	1065
6.	Date of Initial Criticality	August 8, 1976
7.	Date of Commercial Operation	March 1, 1977
8.	Condenser Cooling Method	Once Through With Forced Draft Cooling Tower
9.	Condenser Cooling Water Source	Tennessee River

Utility & Contractor Information

10.	Licensee	Tennessee Valley Authority
11.	Corporate Address	500A Chestnut Street Tower II Chattanooga, Tennessee 37401
12.	Corporate Contact	H. G. Parris, Manager of Power
13.	Architect/Engineer	TVA
14.	Nuclear Steam Supply System	GE
15.	Constructor	TVA

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R.C. Lewis
18.	IE Section Chief	H. C. Dance
19.	IE Resident Inspector	R. Sullivan (205) 729-6196
20.	Licensing Project Manager	R. Clark
21.	Docket Number	50-296
22.	License Number/Date of Issue	DPR-68, 7/2/76
23.	Public Document Room	Athens Public Library South and Forrest Athens, Alabama 35611

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

FULL POWER OPERATIONS.

LAST IE SITE INSPECTION DATE: MARCH 7 - APRIL 4, 1980

INSPECTION REPORT NO: 50-296/80-19

SUMMARY

INSPECTION SUMMARY

INSPECTION JANUARY 14 - FEBRUARY 1 (80-06): THIS ROUTINE INSPECTION INVOLVED 27 RESIDENT INSPECTOR-HOURS IN THE AREAS OF PLANT OPERATIONS, PLANT TOURS, REPORTABLE OCCURRENCES, REFUELING, NONCOMPLIANCE FOLLOWUP, IE BULLETIN FOLLOWUP, PLANT PHYSICAL PROTECTION AND RADIATION AREA CONTROLS. OF THE EIGHT AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION FEBRUARY 26-28 (80-10): THIS SPECIAL, ANNOUNCED INSPECTION INVOLVED 14 INSPECTOR-HOURS ON SITE IN THE AREAS OF SMALL BREAK LOSS OF COOLANT ACCIDENT PROCEDURES AND TRAINING. OF THE TWO AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION MARCH 11-14 (80-11): INCLUDED REVIEW OF THE LICENSEE'S SECURITY PLAN, SECURITY ORGANIZATION, ACCESS CONTROLS TO PROTECTED AND VITAL AREAS, PROTECTED AND VITAL AREA BARRIERS, AND LICENSEE ACTION ON PREVIOUS INSPECTION FINDINGS. THIS SPECIAL INSPECTION WAS IN RESPONSE TO A SERIES OF REPORTED INCIDENTS AT THE BROWNS FERRY NUCLEAR PLANT, BEGINNING IN EARLY FEBRUARY 1980. THE INSPECTION INVOLVED 10 INSPECTOR-HOURS ON SITE. THE INSPECTION WAS BEGUN DURING A REGULAR PERIOD; 2 INSPECTION HOURS WERE ACCOMPLISHED DURING OFFSHIFT PERIODS. THE LICENSEE WAS FOUND TO BE IN COMPLIANCE WITH NRC REQUIREMENTS WITHIN THE AREAS EXAMINED DURING THE INSPECTION.

INSPECTION FEBRUARY 4 - MARCH 7 (80-12): THIS ROUTINE INSPECTION INVOLVED 48 RESIDENT INSPECTOR-HOURS IN THE AREAS OF PLANT OPERATIONS, PLANT TOURS, REPORTABLE OCCURRENCES, REFUELING, PLANT PHYSICAL PROTECTION, AND RADIATION AREA CONTROLS. OF THE SIX AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN FIVE AREAS. ONE ITEM OF APPARENT NONCOMPLIANCE WAS FOUND IN ONE AREA (INFRACTION - FAILURE TO DEMONSTRATE HPCI OPERABLE WHEN RCIC WAS INOPERATIVE FOR MAINTENANCE).

INSPECTION MARCH 19-20 (80-13): THIS SPECIAL ANNOUNCED INSPECTION INVOLVED 4 INSPECTOR-HOURS IN THE AREA OF SEISMIC ANALYSIS FOR AS-BUILT SAFETY-RELATED PIPING SYSTEMS (IE BULLETIN 79-14). NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION MARCH 24-28 AND MARCH 31 - APRIL 4 (80-14): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 22 INSPECTOR-HOURS ON SITE AND 5 INSPECTOR-HOURS OFF SITE IN THE AREAS OF COORDINATION WITH OUTSIDE SUPPORT AGENCIES; EMERGENCY FACILITIES, EQUIPMENT AND PROCEDURES; EMERGENCY TRAINING; EMERGENCY PLANNING AUDITS; EMERGENCY DRILLS; FOLLOWUP ON PREVIOUSLY IDENTIFIED INSPECTION FINDINGS AND FOLLOWUP ON IE BULLETINS. OF THE SEVEN AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN SIX AREAS; TWO ITEMS OF NONCOMPLIANCE WERE FOUND IN ONE AREA (INFRACTION - FAILURE TO PREPARE AND ADHERE TO WRITTEN PROCEDURES; AND INFRACTION - FAILURE TO CONDUCT EMERGENCY TRAINING FOR ALL DESIGNATED EMERGENCY DIRECTORS).

INSPECTION MARCH 7 - APRIL 4 (80-15): THIS ROUTINE INSPECTION INVOLVED 42 RESIDENT INSPECTOR-HOURS IN THE AREAS OF OPERATIONAL SAFETY, MAINTENANCE, REPORTABLE OCCURRENCES, FOLLOWUP ON NONCOMPLIANCE AND UNRESOLVED ITEMS, RADIATION CONTROL, PLANT PHYSICAL PROTECTION AND REACTOR TRIPS. OF THE SEVEN AREAS INSPECTED NO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED IN SIX AREAS. ONE ITEM OF APPARENT NONCOMPLIANCE WAS FOUND IN ONE AREA (INFRACTION - FAILURE TO FOLLOW PROCEDURES IN HEAT-TREATMENT OF UNIT HPCI 1 STEAM PIPING).

REPORTS RECEIVED FROM LICENSEE

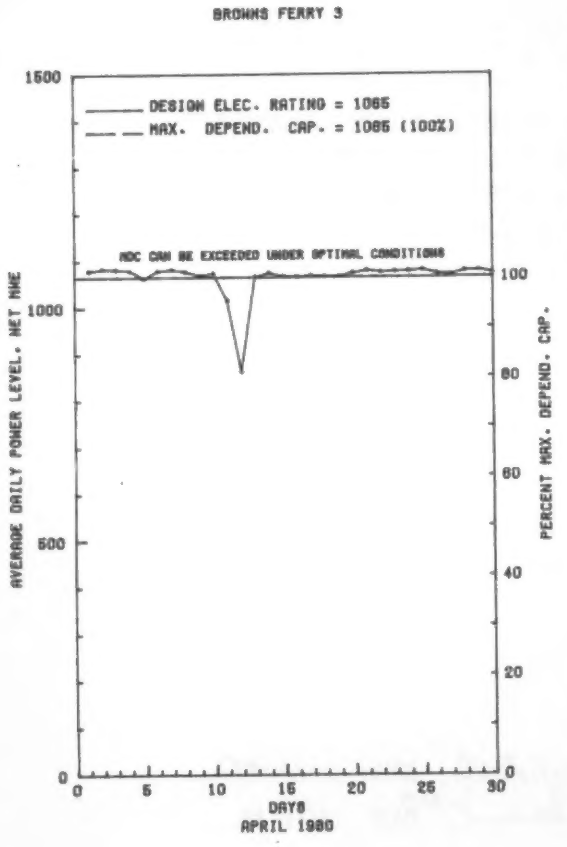
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
80-06/ 03L-0	02/17/80	03/13/80	30 DAY LER	RESIDUAL HEAT REMOVAL SEAL COOLERS FAIL FLOW TEST.

REVIEWED BY OMPA LOEB: Earl Hill **DATE:** 6/12/80

REVIEWED BY NRR DOR: Dick Clark **DATE:** 06-13-80

BROWNS FERRY 3

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-296
2. Utility Contact: Don Green (205) 729-6846
3. Licensed Thermal Power (MWt): 3293
4. Nameplate Rating (Gross MWe): 1280 x 0.9 = 1152
5. Design Electrical Rating (Net MWe): 1065
6. Maximum Dependable Capacity (Gross MWe): 1098
7. Maximum Dependable Capacity (Net MWe): 1065
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	719.0	2,903.9	27,767.0
12. Hours Reactor Was Critical	719.0	2,491.4	21,399.1
13. Reactor Reserve Shutdown Hours	.0	.0	1,658.4
14. Hours Generator On-Line	719.0	2,467.9	20,985.7
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	2,319,562	7,807,463	60,915,309
17. Gross Electrical Energy (MWH)	783,420	2,638,210	20,040,790
18. Net Electrical Energy (MWH)	765,935	2,572,441	19,460,214
19. Unit Service Factor	100.0	85.0	75.3
20. Unit Availability Factor	100.0	85.0	75.3
*21. Unit Capacity Factor (MDC Net)	100.0	83.2	65.8
22. Unit Capacity Factor (DER Net)	100.0	83.2	65.8
23. Unit Forced Outage Rate	.0	15.0	9.8

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUEL OUTAGE - OCTOBER 1980.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
120	04/05/80	S	.0	B	5				DERATED TO PERFORM TURBINE CONTROL VALVE TESTS AND SI'S.
121	04/09/80	F	.0	A	5				DERATED DUE TO RIVER TEMPERATURE SENSOR MALFUNCTION (FALSE HIGH INDICATION).
122	04/11/80	S	.0	H	5				DERATED FOR CONTROL ROD SEQUENCE EXCHANGE FROM "B" TO "A."

SUMMARY

*OPERATION WAS AT NEAR FULL POWER DURING THE REPORT PERIOD, WITH MINOR LOAD REDUCTIONS FOR TESTING AND MAINTENANCE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

BRUNSWICK 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	North Carolina/Brunswick 3 miles N of Southport
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2436
5.	Design Electrical Rating (MWE Net)	821
6.	Date of Initial Criticality	October 8, 1976
7.	Date of Commercial Operation	March 18, 1977
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Cape Fear River

Utility & Contractor Information

10.	Licensee	Carolina Power and Light Company
11.	Corporate Address	411 Fayetteville Street Raleigh, North Carolina 27602
12.	Corporate Contact	J. A. Jones Executive Vice President
13.	Architect/Engineer	United Engineers & Constructors, Inc.
14.	Nuclear Steam Supply System	GE
15.	Constructor	Brown and Root, Inc.

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R.C. Lewis
18.	IE Section Chief	P.J. Kellogg
19.	IE Resident Inspector	J. E. Outzs (919) 457-9531
20.	Licensing Project Manager	J. Hannon
21.	Docket Number	50-325
22.	License Number/Date of Issue	DPR-71, 9/8/76
23.	Public Document Room	Southport-Brunswick County Library 109 W. Moore Street Southport, North Carolina 28461

INSPECTION STATUS

ENFORCEMENT STATUS

THE PLANT NUCLEAR SAFETY COMMITTEE DID NOT PERFORM THE REQUIRED INVESTIGATION OF THE MISSING TEST DATA FOR THE MAY 1979 TESTS OF THE FIRE ALARM SYSTEMS FOR THE SERVICE WATER INTAKE STRUCTURE AND CONTROL BUILDINGS AS REQUIRED BY SECTION 6.5.1.6.E OF THE TECHNICAL SPECIFICATION.
(8011 INFRACTION)

SURVEILLANCE TEST PROCEDURES DO NOT REQUIRE INSPECTION OF ALL FIRE PROTECTION CONTROL VALVES, DIESEL ENGINE FOR DIESEL DRIVEN FIRE PUMP AND ALL OF THE FIRE BARRIER PENETRATIONS AS REQUIRED BY SECTIONS 6.8.1.C, 6.8.1.F, 4.7.7.1.1.C AND 4.7.7.1.2.C OF THE TECHNICAL SPECIFICATIONS.
(8011 INFRACTION)

THE ARRANGEMENT OF THE WATER FLOW ALARM DEVICES FOR THE STANDBY GAS TREATMENT DELUGE SYSTEMS DO NOT COMPLY WITH THE CRITERIA OF NFPA-13 AS REQUIRED BY SECTION IV.C.3.C OF THE FIRE PROTECTION PROGRAM REVIEW.
(8011 DEVIATION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

UNIT IN NORMAL POWER OPERATION.

+ LAST IE SITE INSPECTION DATE: MARCH 31 - APRIL 3, 1980

+ INSPECTION REPORT NO: 50-325/80-15

SUMMARY

INSPECTION SUMMARY

+ INSPECTION MARCH 17-20 (80-10): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 12 INSPECTOR-HOURS ON SITE IN THE AREAS OF INSERVICE INSPECTION WORK OBSERVATION AND DATA REVIEW, INSERVICE INSPECTION PROGRAM REVIEW, TORUS MODIFICATION WORK OBSERVATION. OF THE THREE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION MARCH 31 AND APRIL 1-3 (80-11): THIS SPECIAL, UNANNOUNCED INSPECTION INVOLVED 15 INSPECTOR-HOURS ON SITE IN THE AREAS OF FIRE PROTECTION/PREVENTION. OF THE AREAS INSPECTED, TWO APPARENT ITEMS OF NONCOMPLIANCE WERE FOUND (INFRACTION-INADEQUATE SURVEILLANCE PROCEDURES FOR FIRE PROTECTION SYSTEMS REQUIRED BY TECHNICAL SPECIFICATIONS; AND INFRACTION-INADEQUATE REVIEW OF FIRE PROTECTION TECHNICAL SPECIFICATION VIOLATIONS), AND ONE DEVIATION WAS FOUND (DEVIATION-INADEQUATE WATER FLOW ALARMS FOR STANDBY GAS TREATMENT DELUGE VALVES).

INSPECTION FEBRUARY 29 - MARCH 29, 1980 (80-12): THIS ROUTINE INSPECTION INVOLVED 31.5 INSPECTOR-HOURS ON SITE IN THE AREAS OF CONTROL ROOM OBSERVATIONS, REVIEWS OF OPERATOR LOGS, TAGOUT OF SYSTEMS, TOURS OF VITAL AREAS, PHYSICAL SECURITY, RADIATION PROTECTION, HOUSEKEEPING AND CLEANLINESS, FIRE PROTECTION, AND FOLLOWUP ON PLANT TRIPS. THIS ROUTINE INSPECTION BY THE RESIDENT INSPECTOR INVOLVED 35 MANHOURS OF INSPECTION IN THE NINE AREAS IDENTIFIED ABOVE. OF THE NINE AREAS INSPECTED NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
80-04/ 03L-0	03/08/80	03/25/80	30 DAY LER	CONTAINMENT ATMOSPHERE MONITOR OXYGEN CHANNEL MADE A ONE PERCENT STEP CHANGE W/O A CHANGE IN 02 CONCENTRATION.
80-06/ 03L-0	02/25/80	03/19/80	30 DAY LER	CONTAINMENT ATMOSPHERE MONITOR CAC-AQH-1261 ISOLATION VALVE WOULD NOT SHUT COMPLETELY.
80-12/ 03L-0	02/24/80	03/24/80	30 DAY LER	VALVES X206-DC AND X206DD ON TORUS LEVEL AND PRESSURE TRANSMITTER WENT CLOSE DUE TO LOW AIR PRESSURE.
80-14/ 03L-0	02/14/80	03/13/80	30 DAY LER	SUPPRESSION POOL HIGH LEVEL MANUAL ISOLATION VALVE FOUND CLOSED.
80-15/ 03L-0	03/14/80	03/31/80	30 DAY LER	SWITCH FOR TORUS SUCTION VALVE FAILED WITH VALVE OPEN.
80-16/ 03L-0	02/14/80	03/10/80	30 DAY LER	1D RHR PUMP TRIPPED IN TORUS COOLING MODE MAKING B LOOP OF LOW PRESSURE COOLANT INJECTION INOPERABLE.
80-17/ 03L-0	02/27/80	03/24/80	30 DAY LER	GASEOUS MONITOR ON PRIMARY CONTAINMENT HAD CALIBRATION SHIFT DUE TO MOISTURE IN THE DETECTOR CHAMBER.
80-18/ 03L-0	02/13/80	03/12/80	30 DAY LER	FO-10B VALVE OPEN IN MINIMUM FLOW LINE TO B CORE SPRAY-DRAINED 5500 GALLONS FROM TORUS.
80-21/ 03L-0	03/04/80	04/03/80	30 DAY LER	1C RHR SERVICE WATER PUMP WOULD NOT START DUE TO INSTRUMENT ISOLATION VALVE FOR SUCTION PRESSURE BEING CLOSED.
80-26/ 03L-0	02/12/80	03/10/80	30 DAY LER	BLOWN FUSE IN 28V POWER SUPPLY CAUSED ROD BLOCK ALARMS.

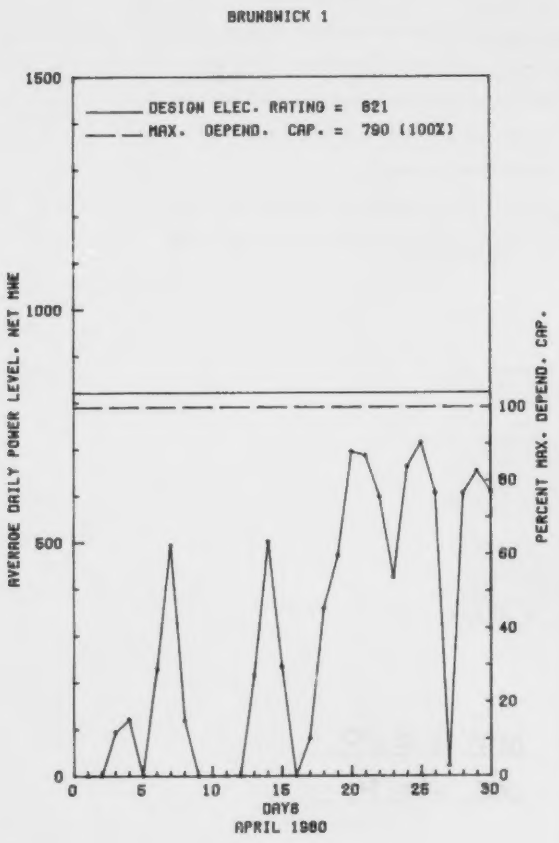
REVIEWED BY OMPA LOEB: ____ **DATE:** 6-12-80

REVIEWED BY NRR DOR: ____ **DATE:** 6/13/80

best copy available

BRUNSWICK 1

AVERAGE POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-325
2. Utility Contact: E. Willis (919) 457-9521
3. Licensed Thermal Power (MWt): 2436
4. Nameplate Rating (Gross MWe): 963 x 0.9 = 867
5. Design Electrical Rating (Net MWe): 821
6. Maximum Dependable Capacity (Gross MWe): 815
7. Maximum Dependable Capacity (Net MWe): 790
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	719.0	2,903.0	27,360.0
12. Hours Reactor Was Critical	516.9	2,655.1	20,044.7
13. Reactor Reserve Shutdown Hours	.0	.0	1,647.1
14. Hours Generator On-Line	443.2	2,533.0	18,865.0
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	689,066	5,292,732	39,122,228
17. Gross Electrical Energy (MWH)	230,580	1,771,230	13,010,049
18. Net Electrical Energy (MWH)	218,417	1,707,732	12,515,542
19. Unit Service Factor	61.6	87.3	69.0
20. Unit Availability Factor	61.6	87.3	69.0
21. Unit Capacity Factor (MDC Net)	38.5	74.5	57.9
22. Unit Capacity Factor (DER Net)	37.0	71.7	55.7
23. Unit Forced Outage Rate	38.4	12.7	20.4

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
REFUELING 05/08/80 - 1920 HOURS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
009	03/31/80	F	50.8	H	4		EC	TRANSF	REACTOR SCRAM ON LOW WATER LEVEL DUE TO AN APPARENT LOSS OF "1D" BUS. THE "1D" BUS FEEDER BREAKER FROM THE START-UP AUXILIARY TRANSFORMER WAS REMOVED FROM THE CABINET FOR PREVENTATIVE MAINTENANCE. THE "1D" BUS FEEDER BREAKER FROM THE UNIT AUXILIARY TRANSFORMER TRIPPED, CAUSING A TOTAL LOSS OF THE 1D BUS.
010	04/03/80	F	42.0	B	5		SF	TURBIN	POWER LIMITED TO APPROXIMATELY 25% OF FULL POWER BECAUSE OF PROBLEMS WITH THE HPCI AUTOMATIC START TEST AND A STEAM LEAK ON HPCI STEAM TRAP BYPASS VALVE.
011	04/05/80	F	30.2	B	3		HJ	VALVEX	REACTOR SCRAMMED AFTER A TURBINE TRIP. THE TURBINE TRIPPED BECAUSE OF HIGH VIBRATION. A CONTROL VALVE PERIODIC TEST WAS IN PROGRESS. THE OUTAGE WAS EXTENDED APPROXIMATELY 16 HOURS DUE TO "INDICATED" LEAKAGE INSIDE THE DRY-WELL EXCEEDING THE AMOUNT OF LEAKAGE ALLOWED BY TECHNICAL SPECIFICATIONS.
012	04/08/80	F	124.6	B	3		EC	ELECON	REACTOR SCRAM FROM A TURBINE TRIP. THE TURBINE TRIP OCCURRED IMMEDIATELY FOLLOWING A "QUICK START" PERIODIC TEST ON #2 DIESEL GENERATOR. AN EXTENSIVE INVESTIGATION REVEALED THE PROBLEM TO BE AN ELECTRICAL GROUND ON #2 DIESEL GENERATOR IN CONJUNCTION WITH THE LOW ELECTRICAL GROUND ON THE DISTRIBUTION SYSTEM.
013	04/15/80	F	50.5	B	3		EC	ELECON	REACTOR SCRAM FROM A TURBINE TRIP. THE TURBINE TRIPPED WHEN STARTING #2 DIESEL GENERATOR. THE PROBLEM WAS A REPLICA OF AN EARLIER OUTAGE (SEE OUTAGE NO. 80-011) WHEN AN ELECTRICAL GROUND ON #2 DIESEL GENERATOR CAUSED A TURBINE TRIP.
014	04/22/80	F	35.0	A	5		HC	HTEXCH	REDUCED POWER TO PERFORM CONDENSER WATERBOX WORK.
015	04/26/80	F	19.7	A	1		SA	VALVEX	UNIT SEPARATED FROM THE GRID BECAUSE OF EXCESSIVE DRYWELL LEAKAGE. AN INVESTIGATION REVEALED A PACKING LEAK ON THE RECIRCULATION LINE SAMPLE VALVE. THE PACKING WAS REPLACED AND ADJUSTMENTS TO THE VALVE WERE MADE.

SUMMARY

THE UNIT REMAINED OFF LINE UNTIL 04/03/80 FOLLOWING LAST MONTH'S TRIP OF THE UNIT AUXILIARY TRANSFORMER FEEDER BREAKER AND SUBSEQUENT REACTOR SCRAM. REACTOR SCRAMS ALSO OCCURRED ON 04/05/80 FOLLOWING A TURBINE TRIP DURING A CONTRO VALVE PERIODIC TEST, AND ON 04/08/80 AND 04/15/80 FROM PROBLEMS WITH ELECTRICAL GROUNDS. EXCESSIVE DRYWELL LEAKAGE ON 04/26/80 REQUIRED REPACKING OF A VALVE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

BRUNSWICK 2

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	North Carolina/Brunswick 3 miles N of Southport
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2436
5.	Design Electrical Rating (MWE Net)	821
6.	Date of Initial Criticality	March 20, 1975
7.	Date of Commercial Operation	November 3, 1975
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Cape Fear River

Utility & Contractor Information

10.	Licensee	Carolina Power and Light Company
11.	Corporate Address	411 Fayetteville Street Raleigh, North Carolina 27602
12.	Corporate Contact	J. A. Jones Executive Vice President
13.	Architect/Engineer	United Engineers & Constructors, Inc.
14.	Nuclear Steam Supply System	GE
15.	Constructor	Brown and Root, Inc.

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R.C. Lewis
18.	IE Section Chief	P.J. Kellogg
19.	IE Resident Inspector	J. E. Outzs (919) 457-9631
20.	Licensing Project Manager	J. Hannon
21.	Docket Number	50-324
22.	License Number/Date of Issue	DPR-62, 12/27/74
23.	Public Document Room	Southport-Brunswick County Library 109 W. Moore Street Southport, North Carolina 28461

INSPECTION STATUS

ENFORCEMENT STATUS

AS REQUIRED BY 10 CFR 50, APPENDIX "B", SECTION V, ACTIVITIES AFFECTING QUALITY SHALL BE PRESCRIBED BY DOCUMENTED INSTRUCTIONS AND PROCEDURES OF A TYPE APPROPRIATE TO THE CIRCUMSTANCES. ANSI N18.7 1972 - AS COMMITTED TO BY THE CAROLINA POWER AND LIGHT COMPANY QUALITY ASSURANCE PROGRAM REQUIRES IN SECTION 5.2.7 THAT MAINTENANCE OF EQUIPMENT SHALL BE PERFORMED IN ACCORDANCE WITH WRITTEN PROCEDURES. CONTRARY TO THE ABOVE MAINTENANCE WAS PERFORMED ON NOVEMBER 29, 1979 FOR UNIT 2 DURING WHICH A FLEXIBLE CONDUIT TO FLOW SWITCH E11-FSL-N021A. (SEE LER 2-79-98 OF DECEMBER 21, 1979) WAS REPLACED WITHOUT AN ADEQUATE PROCEDURE.
(8005 INFRACTION)

AS REQUIRED BY SECTION 7.1.1 - LOCAL AREA EVACUATION OF THE BRUNSWICK STEAM ELECTRIC PLANT - EMERGENCY PLAN, "GENERALLY, PERSONNEL SHALL EVACUATE THE LOCAL AREA WHERE AN AREA RADIATION ALARM OR AIRBORNE ACTIVITY ALARM IS RECEIVED AND CONSIDERED ABNORMAL AND UNEXPLAINED BY OPERATIONS PERSONNEL . . . PERSONNEL NOT MEMBERS OF THE SHIFT OPERATIONS CREW SHALL EVACUATE TO THE PRIMARY ASSEMBLY AREA, WHERE A HEAD COUNT OF PERSONNEL ACCOUNTING WILL BE MADE, DOSE DETERMINATIONS MADE AND PERSONNEL MONITORED FOR CONTAMINATION AS NECESSARY." CONTRARY TO THE ABOVE, ON FEBRUARY 14, 1980, FOLLOWING UNIT 2 REACTOR TRIP AND EVACUATION OF THE REACTOR BUILDING, A HEAD COUNT OF PERSONNEL IN THE REACTOR BUILDING WAS NOT PROMPTLY MADE AS REQUIRED BY THE EMERGENCY PLAN, AND SOME PERSONNEL REMAINED IN THE REACTOR BUILDING FOR AS LONG AS FORTY-THREE MINUTES FOLLOWING THE EVACUATION.
(8008 INFRACTION)

THE PLANT NUCLEAR SAFETY COMMITTEE DID NOT PERFORM THE REQUIRED INVESTIGATION OF THE MISSING TEST DATA FOR THE MAY 1979 TESTS OF THE FIRE ALARM SYSTEMS FOR THE SERVICE WATER INTAKE STRUCTURE AND CONTROL BUILDINGS AS REQUIRED BY SECTION 6.5.1.6.E OF THE TECHNICAL SPECIFICATION.
(8010 INFRACTION)

SURVEILLANCE TEST PROCEDURES DO NOT REQUIRE INSPECTION OF ALL FIRE PROTECTION CONTROL VALVES, DIESEL ENGINE FOR DIESEL DRIVEN FIRE PUMP AND ALL OF THE FIRE BARRIER PENETRATIONS AS REQUIRED BY SECTIONS 6.8.1.C, 6.8.1.F, 4.7.7.1.1.C AND 4.7.7.1.2.C OF THE TECHNICAL SPECIFICATIONS.
(8010 INFRACTION)

THE ARRANGEMENT OF THE WATER FLOW ALARM DEVICES FOR THE STANDBY GAS TREATMENT DELUGE SYSTEMS DO NOT COMPLY WITH THE CRITERIA OF NFPA-13 AS REQUIRED BY SECTION IV.C.3.C OF THE FIRE PROTECTION PROGRAM REVIEW.
(8010 DEVIATION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

UNIT SHUTDOWN FOR REFUELING AND TORUS MODIFICATION.

- LAST IE SITE INSPECTION DATE: MARCH 31 - APRIL 3, 1980
- INSPECTION REPORT NO: 50-324/80-10

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
80-23/ 01P-0	03/19/80	03/19/80	14 DAY LER	LINEAR INDICATION IN TEST CONNECTION ON 2B RECIRCULATION PIPE.
80-26/ 03L-0	02/25/80	03/13/80	30 DAY LER	HPCI STEAM LINE DIFFERENTIAL - MAINSTEAM LINE HIGH FLOW OUT OF CALIBRATION.
80-28/ 03L-0	02/12/80	03/12/80	30 DAY LER	RCIC EXHAUST DIAPHRAM PRESSURE SWITCH OUT OF CALIBRATION 11.9 PSIG VERSUS 10.0 PSIG.
80-29/ 03L-0	02/13/80	03/11/80	30 DAY LER	SUPPRESSION POOL COULD NOT BE PUMPED DOWN BECAUSE RHR VALVE TO RAD WASTE COULD NOT BE OPENED DUE TO BURNED RELAY.
80-32/ 03L-0	02/16/80	03/13/80	30 DAY LER	CONTAINMENT ATMOSPHERE DILUTION TANK BELOW TECH SPEC LIMIT.
80-33/ 03L-0	02/15/80	02/20/80	30 DAY LER	BREAKER TRIPPED WHILE ATTEMPTING TO START 2A RHR ROOM COOLER.
80-34/ 03L-0	03/07/80	04/04/80	30 DAY LER	THREE SNUBBERS FOUND INOPERABLE.
80-35/ 03L-0	02/19/80	03/17/80	30 DAY LER	CONTAINMENT ATMOSPHERIC MONITOR WOULD NOT RUN DUE TO LOW FLOW NOR WOULD INSTRUMENT CALIBRATE.
80-38/ 03L-0	02/29/80	03/27/80	30 DAY LER	2A RECIRC PUMP MG SET TRIPPED DURING CONNECTION OF A WIRE TO CONTACT 3 ON RELAY 52AX.

SUMMARY

INSPECTION SUMMARY

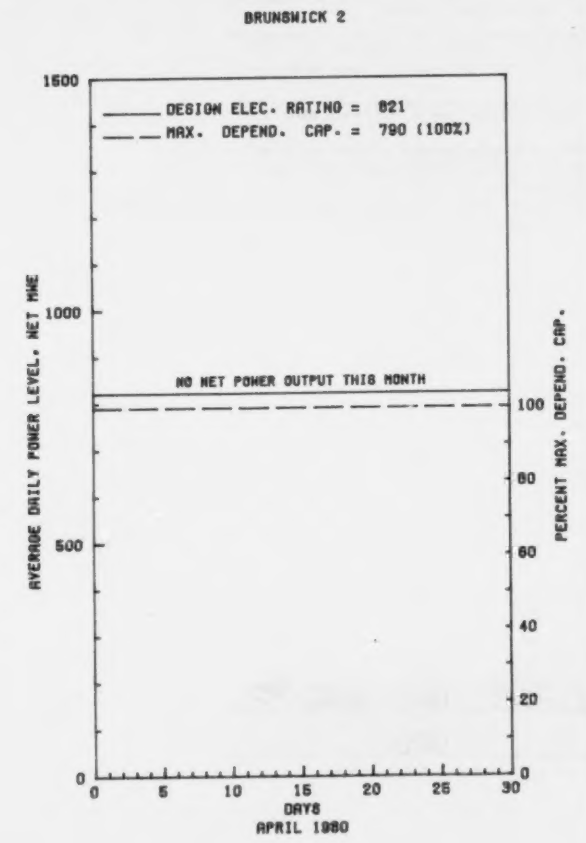
- INSPECTION MARCH 17-20 (80-09): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 12 INSPECTOR-HOURS ON SITE IN THE AREAS OF INSERVICE INSPECTION WORK OBSERVATION AND DATA REVIEW, INSERVICE INSPECTION PROGRAM REVIEW, TORUS MODIFICATION WORK OBSERVATION. OF THE THREE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.
- INSPECTION MARCH 31 - APRIL 1-3 (80-10): THIS SPECIAL, UNANNOUNCED INSPECTION INVOLVED 14 INSPECTOR-HOURS ON SITE IN THE AREAS OF FIRE PROTECTION/PREVENTION. OF THE AREAS INSPECTED, TWO APPARENT ITEMS OF NONCOMPLIANCE WERE FOUND (INFRACTION-INADEQUATE SURVEILLANCE PROCEDURES FOR FIRE PROTECTION SYSTEMS REQUIRED BY TECHNICAL SPECIFICATIONS; AND INFRACTION-INADEQUATE REVIEW OF FIRE PROTECTION TECHNICAL SPECIFICATION VIOLATIONS), AND ONE DEVIATION WAS FOUND (DEVIATION-INADEQUATE WATER FLOW ALARMS FOR STANDBY GAS TREATMENT DELUGE VALVES).
- INSPECTION FEBRUARY 29 - MARCH 29 (80-12): THIS ROUTINE INSPECTION INVOLVED 31.5 INSPECTOR-HOURS ON SITE IN THE AREAS OF CONTROL ROOM OBSERVATIONS, REVIEW OF OPERATOR LOGS, TAGOUT OF SYSTEMS, TOURS OF VITAL AREAS, PHYSICAL SECURITY, RADIATION PROTECTION, HOUSEKEEPING AND CLEANLINESS, FIRE PROTECTION, AND FOLLOWUP ON PLANT TRIPS. ROUTINE AND REACTIVE INSPECTION BY THE RESIDENT INSPECTOR INVOLVED 28 MANHOURS OF INSPECTION TIME ON SITE IN THE AREAS IDENTIFIED ABOVE. OF THE NINE AREAS INSPECTED NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

REVIEWED BY OMPA LOEB: Earl Hill **DATE:** 6-12-80

REVIEWED BY NRR DOR: RB Benson for JNR **DATE:** 6/13/80

BRUNSWICK 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-324
2. Utility Contact: E. Willis (919) 457-9521
3. L'censed Thermal Power (MWt): 2436
4. Nameplate Rating (Gross MWe): 963 x 0.9 = 867
5. Design Electrical Rating (Net MWe): 821
6. Maximum Dependable Capacity (Gross MWe): 815
7. Maximum Dependable Capacity (Net MWe): 790
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	719.0	2,903.0	39,384.0
12. Hours Reactor Was Critical	.0	1,363.7	26,689.6
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	.0	1,228.1	25,102.1
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	2,604,960	46,339,422
17. Gross Electrical Energy (MWH)	0	872,615	15,487,687
18. Net Electrical Energy (MWH)	0	842,126	14,882,598
19. Unit Service Factor	.0	42.3	63.7
20. Unit Availability Factor	.0	42.3	63.7
21. Unit Capacity Factor (MDC Net)	.0	36.7	47.8
22. Unit Capacity Factor (DER Net)	.0	35.3	46.0
23. Unit Forced Outage Rate	.0	8.1	12.8

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 05/22/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	03/01/80	S	719.0	C	4		RC	FUELXX	REFUELING CONTINUES.

SUMMARY

THE UNIT REMAINED SHUTDOWN THROUGHOUT THE REPORT PERIOD FOR REFUELING.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

CALVERT CLIFFS 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Maryland/Calvert 40 miles S of Annapolis
2.	Electric Reliability Council	Mid-Atlantic Area Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2700
5.	Design Electrical Rating (MWE Net)	845
6.	Date of Initial Criticality	October 7, 1974
7.	Date of Commercial Operation	May 8, 1975
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Chesapeake Bay

Utility & Contractor Information

10.	Licensee	Baltimore Gas and Electric Company
11.	Corporate Address	Gas and Electric Building Baltimore, Maryland 21203
12.	Corporate Contact	A. E. Lundvall, Jr. Vice President-Supply
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Combustion Engineering
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Section Chief	E. C. McCabe, Jr.
19.	IE Resident Inspector	R. Architzel (301) 586-2626
20.	Licensing Project Manager	E. L. Conner, Jr.
21.	Docket Number	50-317
22.	License Number/Date of Issue	DPR-53, 7/31/74
23.	Public Document Room	Calvert County Library Prince Frederick, Maryland 20678

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

+ FACILITY ITEMS (PLANS AND PROCEDURES:

+ A modification to increase spent fuel pool storage capacity (Unit No. 1 side) from 200 to 830 racks is planned for the summer and fall of 1980.

PLANT STATUS:

+ No significant new information.

+ LAST IE SITE INSPECTION DATE: 5/6/80

+ INSPECTION REPORT NO: 50-317/80-06

SUMMARY

INSPECTION SUMMARY

+ 50-317/80-01 - Feb 4-29: Routine resident inspections (16 hr. onsite, 4 hr. at the Prince Frederick Fire Department) of plant operations including: tours of the facility; log and record reviews; review of licensee events; IE Bulletins; periodic reports; review of plant operations and Safety Review Committee activities; licensee action on previous inspection findings; and, attendance at the NRC/FEMA/BG&E and Public Meeting on Emergency Planning on February 28, 1980. Noncompliances: Two (PORSC failure to review Technical Specification violations identified in NRC inspection reports; procedures inadequate to prevent the shutdown of a diesel generator with a startup signal present.)

+ 50-317/80-02 - Feb 12-15: Investigation of two minor, unplanned gaseous radiation releases on February 11-12, 1980, and of reasons for non-reporting of these releases. The investigation was conducted by the resident inspector (10 hrs.) and 3 region-based personnel (46 hrs.) including a Section Chief, a Radiation Specialist, and an Investigation Specialist. Noncompliances: None for Unit 1.

+ 50-317/80-03 - Mar 10-28: Routine, onsite regular and backshift inspection by the resident inspector (16 hours). Areas inspected included the control room and the accessible portions of the auxiliary, turbine, service, and intake buildings; radiation protection; physical security; fire protection; plant operating records; and reporting to the NRC. Noncompliances: None.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

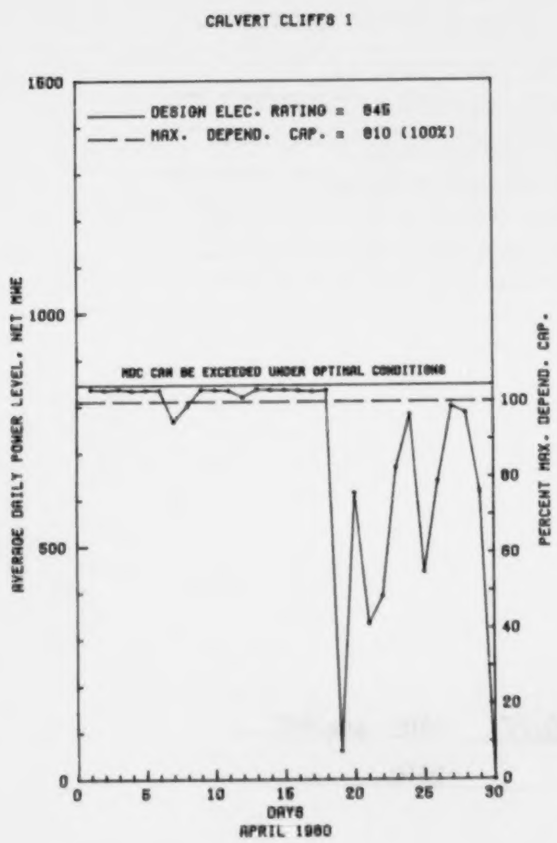
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-002/ 04T	03/22/80	04/03/80	Hourly average condenser temperature rise in excess of limit for 2 hours due to high winds causing low tides, high waves, seaweed in the condenser, and operator inattention
80-013/ 03L	02/25/80	03/25/80	During surveillance 1 channel for containment high pressure tripped outside required value due to a failed signal isolator
80-014/ 03L	03/10/80	04/10/80	During surveillance two sample valves failed to shut on a SI signal due to output relay terminal corrosion. Channel was restored and tested
80-015/ 03L	03/17/80	04/17/80	No. 11 SG aux feed pump OOS for 7.5 hr to repair oil level sightglass
80-020/ 03L	03/14/80	04/14/80	Fire barrier penetration between cable spreading room and turbine bldg not sealed with flamastic fire retardant due to inadequate quality of application
80-021/ 04T	04/04/80	04/16/80	Weekly fuel oil storage tank condensate drainage resulted in oil spill to Chesapeake Bay due to operator error and loss of water seal in oil interceptor. Corrective action includes scheduling evolution during daylight and draining condensate to a pail

REVIEWED BY OMPA LOEB: Earl Hill **DATE:** 6/22/80

REVIEWED BY NRR DOR: ______ **DATE:** ______

CALVERT CLIFFS 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-317
2. Utility Contact: S. D. Merson (301) 234-5243
3. Licensed Thermal Power (MWt): 2700
4. Nameplate Rating (Gross MWe): 1020 x 0.9 = 918
5. Design Electrical Rating (Net MWe): 845
6. Maximum Dependable Capacity (Gross MWe): 845
7. Maximum Dependable Capacity (Net MWe): 810
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	719.0	2,903.0	43,668.0
12. Hours Reactor Was Critical	672.4	2,446.9	34,886.5
13. Reactor Reserve Shutdown Hours	11.4	19.9	1,079.5
14. Hours Generator On-Line	645.8	2,384.0	34,086.0
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,644,922	5,167,418	80,484,486
17. Gross Electrical Energy (MWH)	529,683	1,621,465	26,433,995
18. Net Electrical Energy (MWH)	504,622	1,531,889	25,181,095
19. Unit Service Factor	89.8	82.1	78.1
20. Unit Availability Factor	89.8	82.1	78.1
*21. Unit Capacity Factor (MDC Net)	86.6	65.1	71.2
22. Unit Capacity Factor (DER Net)	83.1	62.4	68.2
23. Unit Forced Outage Rate	6.1	3.1	8.6

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
REFUELING, INSPEC. & TMI - MODS - 10/17 - 12/15/80
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-04	04/19/80	S	19.5	B	1		ZZ	VALVEX	TO REPLACE A HYDRAULIC CONTROL VALVE FOR #1 MAIN TURBINE INTERCEPT VALVE.
80-05	04/21/80	F	15.5	A	3		ZZ	CRDRVE	VOLTAGE SWINGS ON THE CONTROL ELEMENT DRIVE SYSTEM MOTOR GENERATOR SETS CAUSED A TURBINE/REACTOR TRIP.
80-06	04/25/80	F	13.8	A	3		ZZ	CRDRVE	TRIP DUE TO UNDERVOLTAGE TO THE REACTOR TRIP BREAKERS WHILE TROUBLE SHOOTING ABOVE REASON (80-05).
80-07	04/29/80	S	11.4	B	1		ZZ	VALVEX	TO EFFECT REPAIRS ON MAIN TURBINE INTERCEPT VALVES #1 & #4.
80-08	04/30/80	F	13.0	A	9		ZZ	PUMPXX	LEAK ON #11B REACTOR COOLANT PUMP CONTROL BLEED - OFF LINE. PLANT WAS ALREADY OFF FOR ABOVE OUTAGE (80-07)/

SUMMARY

OPERATION WAS ROUTINE DURING THE REPORT PERIOD EXCEPT TWO TRIPS. A TURBINE/REACTOR TRIP OCCURRED ON 04/21/80 FROM VOLTAGE SWINGS ON THE CONTROL ELEMENT ASSEMBLY DRIVE SYSTEM MOTOR GENERATOR SETS, AND THE REACTOR TRIPPED ON 04/25/80 WHILE TROUBLE SHOOTING THESE SETS. LOAD REDUCTIONS WERE NECESSARY TO REPAIR VALVES AND A REACTOR COOLANT PUMP LEAK.

LEGEND

[1] Type:
ed
duled

[2] Reason:
A–Equipment Failure
B–Maintenance or Test
C–Refueling
D–Regulatory Restriction
E–Operator Training & License Examination
F–Administrative
G–Operational Error
H–Other

[3] Method:
1–Manual
2–Manual Scram
3–Automatic Scram
4–Continuations
5–Load Reduction
9–Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

CALVERT CLIFFS 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1. Location (state/county) (nearest population center)	Maryland/Calvert 40 miles S of Annapolis
2. Electric Reliability Council	Mid-Atlantic Area Council
3. Type of Reactor	PWR
4. Power Level (MWT)	2700
5. Design Electrical Rating (MWE Net)	845
6. Date of Initial Criticality	November 30, 1976
7. Date of Commercial Operation	April 1, 1977
8. Condenser Cooling Method	Once Through
9. Condenser Cooling Water Source	Chesapeake Bay

Utility & Contractor Information

10. Licensee	Baltimore Gas and Electric Company
11. Corporate Address	Gas and Electric Building Baltimore, Maryland 21203
12. Corporate Contact	A. E. Lundvall, Jr. Vice President-Supply
13. Architect/Engineer	Bechtel
14. Nuclear Steam Supply System	Combustion Engineering
15. Constructor	Bechtel

Regulatory Information

16. IE Region responsible	I
17. IE Branch Chief	E. Brunner
18. I E Section Chief	E. C. MCCABE, JR.
19. IE Resident Inspector	R. Architzel (301) 586-2626
20. Licensing Project Manager	E. L. Conner, Jr.
21. Docket Number	50-318
22. License Number/Date of Issue	DPR-69, 8/13/76
23. Public Document Room	Calvert County Library Prince Frederick, Maryland 20678

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS:

- No significant new information.
- LAST IE SITE INSPECTION DATE: 5/6/80
- INSPECTION REPORT NO: 50-318/80-06

SUMMARY

INSPECTION SUMMARY

- 50-318/80-01 - Feb 4-29: Routine resident inspections (16 hr. onsite, 5 hr. at the Prince Frederick Fire Department) of plant operations including: tours of the facility; log and record reviews; review of licensee events; IE Bulletins; periodic reports; review of plant operations and Safety Review Committee activities; licensee action on previous inspection findings; and, attendance at the NRC/FEMA/BG&E and Public Meeting on Emergency Planning on February 28, 1980. Noncompliances: Three (Failure to make a prompt report following the simul- taneous inoperability of all Unit 2 Diesel Generators; PORSC failure to review Technical Specification violations identi- fied in NRC inspection reports; procedures inadequate to prevent the shutdown of a Diesel Generator with a starting signal present).
- 50-318/80-02 - Feb 12-15: Investigation of two minor, unplanned gaseous radiation releases on February 11-12, 1980, and of reasons for non-reporting of those releases. The investigation was conducted by the resident inspector (20 hrs.) and 3 region-based personnel (70 hrs.) including a Section Chief, a Radiation Specialist and an Investigation Specialist. Noncompliances: Six (Failure to notify the NRC; failure to estimate offsite dose; failure to brief individual making re-entry into evacuated area; failure to follow procedure requiring valve isolation of ion exchanger; failure to follow procedure for placing ion exchanger in service; and, failure to have appropriate procedure sequence for isolating ion exchanger).
- 50-318/80-03 - Mar 10-28: Routine, onsite regular and backshift inspection by the resident inspector (16 hours). Areas inspected included the control room and the accessible portions of the auxiliary, turbine, service, and intake buildings; radiation protection; physical security; fire protection; plant operating records; and reporting to the NRC. Noncompliances: None.

REPORTS RECEIVED FROM LICENSEE

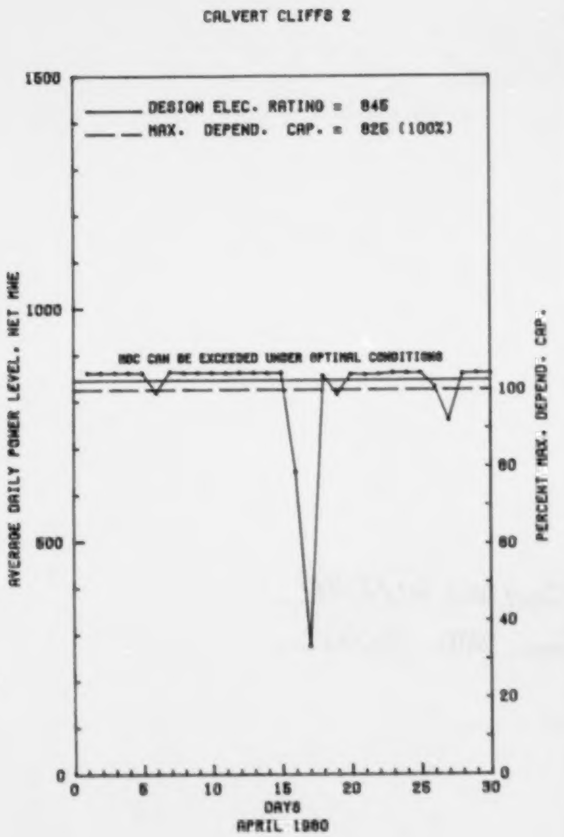
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-001/ 03L	02/25/80	03/25/80	Disassembly of #21 containment spray pump instead of #21 low pressure safety injection pump for corrective maintenance due to administrative error
80-010/ 03L	02/24/80	03/24/80	Faulty control element assembly (CEA) 31 and CEA 34 and subsequent drop into the core during surveillance testing
80-013/ 03L	03/03/80	04/02/80	Channel EZF pressure input signal for #22 SG isolation was bypassed and replaced, remaining channels were operable. The signal isolator was found out of calibration
80-015/ 03L	02/27/80	03/25/80	Accidental grounding of the prime 15V power supply to CEA 44 during maintenance and subsequent drop of CEA 44 into the core
80-016/ 03L	03/03/90	03/28/80	An emergency diesel generator was placed out of service for failure to reach rated speed in less than 10 seconds due to governor slow response
80-017/ 03L	03/16/80	04/10/80	The No. 22 saltwater/service water heat exchanger was taken out of service on 3/16 to repair a tube leak. Redundant loop was inservice. Tube was plugged and heat exchanger was returned to service
80-018/ 03L	03/11/80	04/10/80	Leaking weld on 23 charging pump suction relief valve due to vibration induced metal fatigue. Modifications planned for permanent fix
80-019/ 03L	03/12/80	03/25/80	Separation criteria violated in Cable Spreading Room by incorrect routing of cable #A2Y02929P
80-020/ 03L	03/20/80	04/18/80	Repetitive, but intermittent channel D RPS trip condition from undeter- mined cause, with channel bypassed during its intermittent trip periods
80-021/ 03L	03/18/80	04/18/80	No. 23 saltwater pump start failure during surveillance due to faulty relay socket

REVIEWED BY OMPA LOEB: Earl Hull DATE: 6/12/80

REVIEWED BY NRR DOR:__________ DATE:________

CALVERT CLIFFS 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-318
2. Utility Contact: S. D. Merson (301) 234-5228
3. Licensed Thermal Power (MWt): 2700
4. Nameplate Rating (Gross MWe): 1012 x 0.9 = 911
5. Design Electrical Rating (Net MWe): 845
6. Maximum Dependable Capacity (Gross MWe): 860
7. Maximum Dependable Capacity (Net MWe): 825
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

ITEM 21 CALCULATED BY USING A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	719.0	2,903.0	27,023.0
12. Hours Reactor Was Critical	708.4	2,673.3	22,610.2
13. Reactor Reserve Shutdown Hours	.0	.0	413.1
14. Hours Generator On-Line	701.4	2,658.1	22,266.6
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,835,790	6,871,633	55,115,865
17. Gross Electrical Energy (MWH)	621,966	2,313,697	18,318,920
18. Net Electrical Energy (MWH)	595,566	2,213,473	17,470,493
19. Unit Service Factor	97.6	91.6	82.4
20. Unit Availability Factor	97.6	91.6	82.4
*21. Unit Capacity Factor (MDC Net)	100.4	92.4	78.7
22. Unit Capacity Factor (DER Net)	98.0	90.2	76.5
23. Unit Forced Outage Rate	2.4	.7	6.2

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-01	04/16/80	F	17.6	A	1		ZZ	VALVEX	TO REPAIR A FAILED LETDOWN CONTROL VALVE.

SUMMARY

OPERATION WAS AT NEAR FULL POWER DURING THE REPORT PERIOD. THE UNIT WAS TAKEN OFF LINE ON 04/16/80 TO REPAIR A FAILED LETDOWN CONTROL VALVE.

LEGEND

[1] Type:
F–Forced
S–Scheduled

[2] Reason:
A–Equipment Failure
B–Maintenance or Test
C–Refueling
D–Regulatory Restriction
E–Operator Training & License Examination
F–Administrative
G–Operational Error
H–Other

[3] Method:
1–Manual
2–Manual Scram
3–Automatic Scram
4–Continuations
5–Load Reduction
9–Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

NRC POWER RESTRICTIONS: NONE

COOK 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Michigan/Berrien 11 miles S of Benton Harbor
2.	Electric Reliability Council	East Central Area Reliability Coordination Agreement
3.	Type of Reactor	PWR
4.	Power Level (MWT)	3250
5.	Design Electrical Rating (MWE Net)	1054
6.	Date of Initial Criticality	January 18, 1975
7.	Date of Commercial Operation	August 27, 1975
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10.	Licensee	AEP Corporation
11.	Corporate Address	Indiana & Michigan Power Co. 2 Broadway New York, New York 10004
12.	Corporate Contact	John Dolan Senior Executive V. P. Eng.
13.	Architect/Engineer	AEP
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Indiana & Michigan Power Co.

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	D.C. Boyd
19.	IE Resident Inspector	R. Masse (616) 465-6775
20.	Licensing Project Manager	S. Miner
21.	Docket Number	50-315
22.	License Number/Date of Issue	DPR-58 issued 10/25/74
• 23.	Public Document Room	Maude Preston Palenske Memorial Library 500 Market Street St. Joseph, Michigan 49085

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

Operating routinely.

*LAST IE SITE INSPECTION DATE: March 10-14, 1980

* INSPECTION REPORT NO: 80-05

SUMMARY

INSPECTION SUMMARY

* Inspection on March 10-14, (80-05): Included a review of Security Plan; Security Program Audit; Security Organization; Protected Area Physical Barriers; Vital Area Physical Barriers; Lighting; Access Control (Identification, Authorization and Badging); Access Control (Search); Access Control (Escorting); Access Control (Vital Areas); Alarm Stations; Detection Aids; Assessment Aids; Communications; Testing and Maintenance; Compensatory Measures; Power Supply; Response; Locks, Keys, and Combinations; Records and Reports; Followup on Previous Items of Noncompliance/Deviations; Followup on Unresolved Items; and Followup on IE Bulletin No. 79-16. The inspection involved 80 inspector hours onsite by two NRC inspectors. The inspection began during day shift period; 14 inspection hours were accomplished during offshift periods. The licensee was found to be in compliance with NRC requirements within the 23 areas examined during this inspection except for the following three items: Access Control (Identification, Authorization, and Badging) - The licensee failed to conduct proper screening in accordance with security plans commitments; Physical Barriers (Vital Areas) - A vital area barrier was found to be degraded; and Access Control (Identification, Authorization, and Badging) - The licensee failed to implement adequate badge control. The items of noncompliance from the previous security inspection IE Report No. 50-315/79-19; 50-316/79-15 were closed. One unresolved item identified in that inspection remains open.

REPORTS RECEIVED FROM LICENSEE

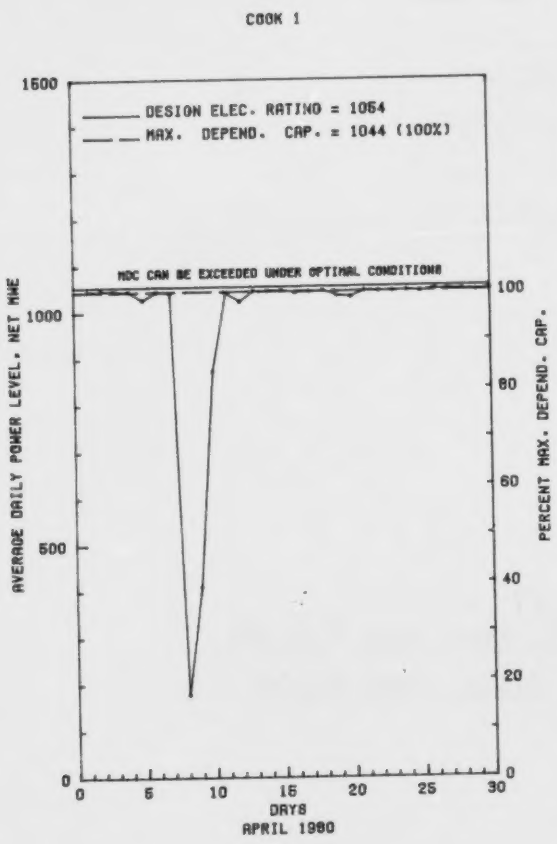
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-04/ 03L-0	03/11/80	04/08/80	While performing a scan sequence one of the two detectors in use for APDMS became inoperable and could not complete its scan.
80-05/ 03L-0	03/21/80	04/18/80	On 3/21/80, it was discovered that the Unit 1 APDMS and P-250 APDMS limits, installed since the previous surveillance were incorrect.
80-06/ 03L-0	03/24/80	04/18/80	During the performance of a surveillance test on the Unit 1 650 foot elevation airlock door a leakage of 130 CFM was found.

REVIEWED BY OMPA LOEB: T.C. Cintula DATE: 6-12-80

REVIEWED BY NRR DOR: DATE: 6/13/80

COOK 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-315
2. Utility Contact: W. T. Gillett (616) 465-5901
3. Licensed Thermal Power (MWt): 3250
4. Nameplate Rating (Gross MWe): 1280 x 0.9 = 1152
5. Design Electrical Rating (Net MWe): 1054
6. Maximum Dependable Capacity (Gross MWe): 1080
7. Maximum Dependable Capacity (Net MWe): 1044
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	719.0	2,903.0	46,727.0
12. Hours Reactor Was Critical	702.6	2,489.4	35,451.1
13. Reactor Reserve Shutdown Hours	.0	.0	463.0
14. Hours Generator On-Line	700.6	2,481.2	34,564.4
15. Unit Reserve Shutdown Hours	.0	.0	321.0
16. Gross Thermal Energy (MWH)	2,212,354	7,813,148	96,479,453
17. Gross Electrical Energy (MWH)	736,280	2,596,960	31,754,300
18. Net Electrical Energy (MWH)	710,617	2,506,194	30,503,508
19. Unit Service Factor	97.4	85.5	77.7
20. Unit Availability Factor	97.4	85.5	77.7
21. Unit Capacity Factor (MDC Net)	94.7	82.7	68.1
22. Unit Capacity Factor (DER Net)	93.8	81.9	63.4
23. Unit Forced Outage Rate	2.6	14.5	7.4

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
MAY 30, 1980 - JULY 20, 1980 - REFUELING OUTAGE.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
159	04/08/80	F	18.4	H	3		ZZ	ZZZZZZ	UNIT TRIPPED FROM 100% AS A RESULT OF A LIGHTNING STRIKE NEAR UNDERGROUND CONTROL CABLES BETWEEN THE PLANT AND THE SWITCHYARDS. GENERATOR OUTPUT BREAKERS OPENED CAUSING LOAD REJECTION. THIS IN TURN CAUSED STEAM GENERATOR WATER LEVELS TO SHRINK BELOW THE LOW-LOW LEVEL TRIP POINT WHICH CAUSED THE REACTOR TRIP.

SUMMARY

OPERATION WAS AT NEAR FULL POWER DURING THE REPORT PERIOD, EXCEPT FOR A UNIT TRIP ON 04/08/80 FROM A LIGHTNING STRIKE WHICH CAUSED THE GENERATOR OUTPUT BREAKERS TO OPEN, RESULTING IN A LOAD REJECTION AND LOW STEAM GENERATOR LEVEL.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

COOK 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1. Location (state/county) (nearest population center)	Michigan/Berrien 11 miles S of Benton Harbor
2. Electric Reliability Council	East Central Area Reliability Coordination Agreement
3. Type of Reactor	PWR
4. Power Level (MWT)	3391
5. Design Electrical Rating (MWE Net)	1100
6. Date of Initial Criticality	March 10, 1978
7. Date of Commercial Operation	July 1, 1978
8. Condenser Cooling Method	Once Through
9. Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10. Licensee	AEP Corporation Indiana & Michigan Power Co. Indiana & Michigan Electric Co.
11. Corporate Address	2 Broadway New York, New York 10004
12. Corporate Contact	John Dolan, VP/Chief Engineer (212) 422 - 4800
13. Architect/Engineer	American Elec. Power Service Corp.
14. Nuclear Steam Supply System	Westinghouse
15. Constructor	Indiana & Michigan Power Co.

Regulatory Information

16. IE Region responsible	III
17. IE Branch Chief	R. Heishman
18. IE Section Chief	D.C. Boyd
19. IE Resident Inspector	R. Masse (616) 465-6775
20. Licensing Project Manager	S. Miner
21. Docket Number	50-316
22. License Number/Date of Issue	DPR-74, 12/23/77
23. Public Document Room	Maude Preston Palenske Memorial Library 500 Market Street St. Joseph, Michigan 49085

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

Operating routinely.

+LAST IE SITE INSPECTION DATE: March 10-14, 1980

+ INSPECTION REPORT NO: 80-04

SUMMARY

INSPECTION SUMMARY

+ Inspection on March 10-14, (80-04): Included a review of Security Plan;
Security Program Audit; Security Organization; Protected Area Physical
Barriers; Vital Area Physical Barriers; Lighting; Access Control (Identi-
fication, Authorization and Badging); Access Control (Search); Access
Control (Escorting); Access Control (Vital Areas); Alarm Stations;
Detection Aids; Assessment Aids; Communications; Testing and Maintenance;
Compensatory Measures; Power Supply; Response; Locks, Keys, and
Combinations; Records and Reports; Followup on Previous Items of Noncompli-
ance/Deviations; Followup on Unresolved Items; and Followup on IE Bulletin
No. 79-16. The inspection involved 80 inspector hours onsite by two NRC
inspectors. The inspection began during day shift period; 14 inspection
hours were accomplished during offshift periods. The licensee was found
to be in compliance with NRC requirements within the 23 areas examined
during this inspection except for the following three items: Access
Control (Identification, Authorization, and Badging) - The licensee failed
to conduct proper screening in accordance with security plans commitments;
Physical Barriers (Vital Areas) - A vital area barrier was found to be
degraded; and Access Control (Identification, Authorization, and Badging) -
The licensee failed to implement adequate badge control. The items of
noncompliance from the previous security inspection IE Report No. 50-315/
79-19; 50-316/79-15 were closed. One unresolved item identified in that
inspection remains open.

REPORTS RECEIVED FROM LICENSEE

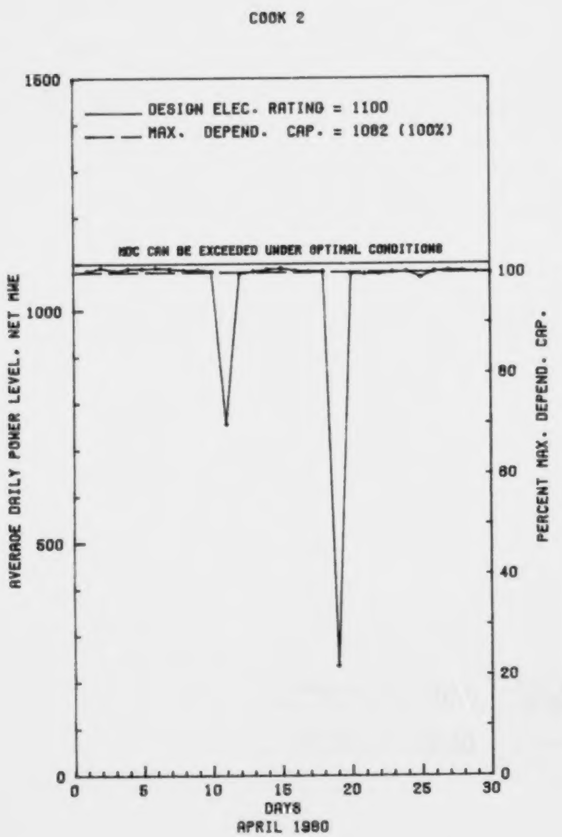
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-12/ 03L-0	03/05/80	03/25/80	A steam generator water level-low-low reactor trip bistable was found to trip at a value corresponding to 11:05% narrow range instrument span.
80-13/ 03L-0	03/07/80	04/02/80	During normal operation, the circuit breaker for the R-11 and R-12 sample pump tripped.
80-14/ 03L-0	02/16/80	04/02/80	During a surveillance test, the proximity switch for "D" main turbine stop valve fell off of the mounting bracket.
80-15/ 03L-0	03/25/80	04/15/80	During an operational retest of the turbine driven auxiliary feed pump, aux feed valve FMO-221 for No. 2 steam generator did not operate.
80-16/ 03L-0	03/22/80	04/18/80	Control room operator discovered the breaker for IMO-310 open.
80-17/ 03L-0	03/21/80	04/18/80	During normal operation, the TDAFP trip and throttle valve tripped and could not be reset.

==

REVIEWED BY OMPA LOEB: T.C. Cintula **DATE:** 6.12.80

REVIEWED BY NRR DOR: **DATE:** 6/13/80

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-316
2. Utility Contact: W. T. Gillett (616) 465-5901
3. Licensed Thermal Power (MWt): 3391
4. Nameplate Rating (Gross MWe): 1333 x 0.85 = 1133
5. Design Electrical Rating (Net MWe): 1100
6. Maximum Dependable Capacity (Gross MWe): 1118
7. Maximum Dependable Capacity (Net MWe): 1082
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	719.0	2,903.0	20,423.0
12. Hours Reactor Was Critical	707.7	2,437.5	13,517.4
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	705.3	2,352.4	12,854.3
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	2,354,329	7,523,166	39,747,546
17. Gross Electrical Energy (MWH)	778,750	2,477,230	12,631,640
18. Net Electrical Energy (MWH)	752,488	2,389,325	12,156,737
19. Unit Service Factor	98.1	81.0	71.7
20. Unit Availability Factor	98.1	81.0	71.7
21. Unit Capacity Factor (MDC Net)	96.7	76.1	65.9
22. Unit Capacity Factor (DER Net)	95.1	74.8	64.8
23. Unit Forced Outage Rate	1.9	3.4	11.8

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
75	04/11/80	F	.0	A	4		CH	MECFUN	REACTOR POWER REDUCED TO 58% TO REMOVE THE WEST MAIN FEED PUMP FROM SERVICE DUE TO FAULTY PUMP-TURBINE COUPLING. REPAIR COMPLETED AND REACTOR POWER RETURNED TO 100% 04/12/80.
76	04/19/80	F	13.7	A	3		CA	VALVOP	REACTOR/TURBINE TRIP DUE TO STEAM FLOW/FEEDWATER FLOW MISMATCH COINCIDENT WITH LOW LEVEL IN NO. 4 STEAM GENERATOR. LOW FEEDWATER FLOW AND STEAM GENERATOR LEVEL WAS DUE TO PROBLEMS WITH NO. 4 STEAM GENERATOR FEEDWATER REGULATING VALVE, WHEN ATTEMPTING TO RECOVER FROM AN AUTOMATIC ISOLATION OF MOISTURE SEPARATOR-REHEATER COIL BUNDLES.

SUMMARY

OPERATION WAS AT NEAR FULL POWER DURING THE REPORT PERIOD, WITH A LOAD REDUCTION ON 04/11/80 TO REPAIR THE PUMP TURBINE COUPLING ON THE WEST MAIN FEED PUMP. A REACTOR (TURBINE TRIP OCCURRED ON 04/19/80 DUE TO STEAM FLOW) FEEDWATER FLOW MISMATCH AND LOW LEVEL IN THE NO. 4 STEAM GENERATOR.

LEGEND

[1] Type:
F–Forced
S–Scheduled

[2] Reason:
A–Equipment Failure
B–Maintenance or Test
C–Refueling
D–Regulatory Restriction
E–Operator Training & License Examination
F–Administrative
G–Operational Error
H–Other

[3] Method:
1–Manual
2–Manual Scram
3–Automatic Scram
4–Continuations
5–Load Reduction
9–Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

COOPER STATION

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Nebraska/Nemaha 23 miles S of Nebraska City
2.	Electric Reliability Council	Mid-Continent Area Reliability Coordination Agreement
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2381
5.	Design Electrical Rating (MWE Net)	778
6.	Date of Initial Criticality	February 21, 1974
7.	Date of Commercial Operation	July 1, 1974 (50% rating)
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Missouri River

Utility & Contractor Information

10.	Licensee	Nebraska Public Power District
11.	Corporate Address	P. O. Box 499 Columbus, Nebraska 68601
12.	Corporate Contact	J. M. Pilant, Director Licensing and Quality Assurance
13.	Architect/Engineer	Burns and Roe, Inc.
14.	Nuclear Steam Supply System	GE
15.	Constructor	Burns and Roe, Inc.

Regulatory Information

16.	IE Region responsible	IV
17.	IE Branch Chief	G. L. Madsen
18.	IE Section Chief	T. F. Westerman
19.	IE Resident Inspector	G. L. Constable
20.	Licensing Project Manager	V. L. Rooney
21.	Docket Number	50-298
22.	License Number/Date of Issue	DPR-46, 1/18/74
23.	Public Document Room	Auburn Public Library 118 15th Street Auburn, Nebraska 68305

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

CRACKS FOUND ON NO. 1 AND 2 LOW PRESSURE TURBINE ROTOR DISC.

FACILITY ITEMS (PLANS AND PROCEDURES):

MANAGERIAL ITEMS:

PLANT STATUS:

SHUT DOWN FOR REFUELING TORUS MODIFICATIONS AND INSPECTION OF WESTINGHOUSE TURBINE FOR CRACKING ON 3/1/80. OUTAGE EXPECTED TO EXTEND TO MAY 31, 1980.

LAST IE SITE INSPECTION DATE: MARCH 11 - 13; 24 - 27, 1980

INSPECTION REPORT NO: 50-298/80-04

SUMMARY

INSPECTION SUMMARY

INSPECTION CONDUCTED MARCH 17-20, 1980 (80-03): FACILITY ORGANIZATION AND OPERATION, MEASUREMENTS AND CONTROLS, INVENTORY AND INVENTORY VERIFICATION, SHIPPING AND RECEIVING, STORAGE AND INTERNAL CONTROLS, RECORDS AND REPORTS AND MANAGEMENT OF MATERIAL CONTROL SYSTEM. THE INSPECTION INVOLVED 38 INSPECTION HOURS ONSITE BY TWO NRC INSPECTORS AND WAS BEGUN DURING THE REGULAR HOURS. THE LICENSEE WAS FOUND TO BE IN COMPLIANCE WITH NRC REQUIREMENTS IN THE 7 AREAS EXAMINED DURING THE INSPECTION.

INSPECTION CONDUCTED MARCH 17-20, 1980 (80-03): INCLUDED REVIEW OF SECURITY PLAN; SECURITY ORGANIZATION; PHYSICAL BARRIERS (PROTECTED AREA); PHYSICAL BARRIERS (VITAL AREAS); ACCESS CONTROL (SEARCH); ACCESS CONTROL (ESCORTING); ALARM STATION; COMMUNICATIONS, ASSESSMENT AIDS; COMPENSATORY MEASURES; POWER SUPPLY; RECORDS AND REPORTS. THE INSPECTION INVOLVED 62 INSPECTOR HOURS ONSITE BY TWO NRC INSPECTORS. THE INSPECTION WAS BEGUN DURING A REGULAR SHIFT WITH TWO INSPECTOR HOURS BEING ACCOMPLISHED OFFSITE AND SIX INSPECTOR HOURS WERE COMPLETED DURING OFFSHIFT PERIODS. THE LICENSEE WAS FOUND TO BE IN COMPLIANCE WITH NRC REQUIREMENTS WITHIN THE 12 AREAS EXAMINED DURING THE INSPECTION.

INSPECTION ON MARCH 11-13, AND 24-27, 1980 (80-04): ROUTINE, ANNOUNCED INSPECTION OF QUALITY ASSURANCE PROGRAM; GENERAL ELECTRIC TYPE SBM CONTROL SWITCH PROBLEM, AND A REVIEW OF REFUELING OUTAGE ACTIVITIES. THE INSPECTION INVOLVED SIXTY-SIX (66) INSPECTOR-HOURS ONSITE BY TWO INSPECTORS. OF THE FOUR (4) AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE NOTED IN ANY AREA.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

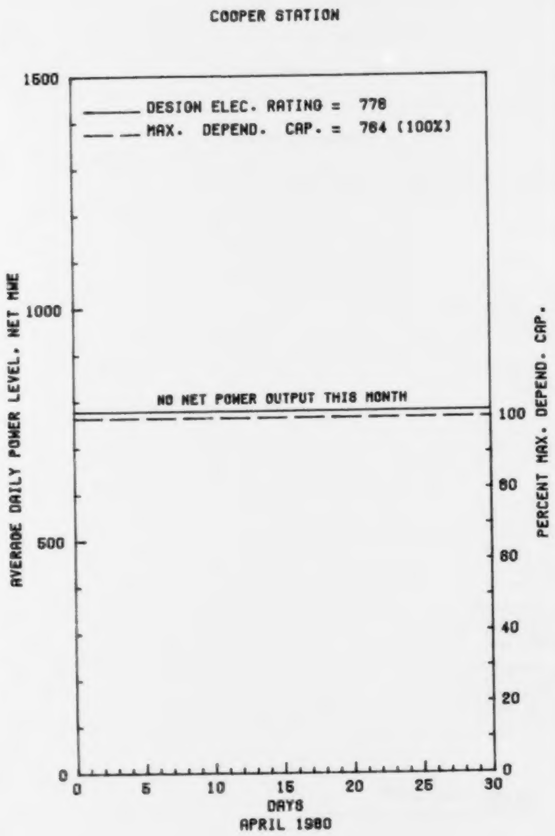
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-01 03T-0	1/2/80	1/11/80	HPCI AUXILIARY OIL PUMP FOUND IN PULL-TO-LOCK
80-02 03L-0	1/3/80	1/21/80	HPCI STOP VALVE STEM FAILED
80-03 03L-0	1/11/80	2/11/80	TORUS ISOLATION VALVE NOT FULLY CLOSED
80-04	1/15/80	2/11/80	RHR MOV FOUND WITH BLOWN FUSE
80-05	1/16/80	2/19/80	PINHOLE LEAKS FOUND IN RECS
80-06	2/22/80	3/24/80	FIRE SPRINKLER DISCHARGED IN RB QUADRANT
80-07	3/10/80	3/25/80	SBLC FLOW FOUND LOW
80-08	3/12/80	4/14/80	RCIC TURBINE BLADE DAMAGE

REVIEWED BY OMPA LOEB: Earl Hell **DATE:** 6-12-80

REVIEWED BY NRR DOR: Dick Clark **DATE:** 06-13-80

COOPER STATION

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number 50-298
2. Utility Contact: P. J. Borer (402) 825-3811
3. Licensed Thermal Power (MWt): 2381
4. Nameplate Rating (Gross MWe): 983 x 0.85 = 836
5. Design Electrical Rating (Net MWe): 778
6. Maximum Dependable Capacity (Gross MWe): 787
7. Maximum Dependable Capacity (Net MWe): 764
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	719.0	2,903.0	51,144.0
12. Hours Reactor Was Critical	.0	1,447.0	42,684.3
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	.0	1,437.7	41,913.3
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	3,077,520	79,260,246
17. Gross Electrical Energy (MWH)	0	1,018,195	25,557,043
18. Net Electrical Energy (MWH)	0	981,128	24,639,405
19. Unit Service Factor	.0	49.5	82.0
20. Unit Availability Factor	.0	49.5	82.0
21. Unit Capacity Factor (MDC Net)	.0	44.2	63.1
22. Unit Capacity Factor (DER Net)	.0	43.4	61.9
23. Unit Forced Outage Rate	.0	1.7	4.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 05/29/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-5	03/01/80	S	719.0	C	4				1980 REFUELING AND MAINTENANCE OUTAGE.

SUMMARY

THE UNIT WAS SHUTDOWN THROUGHOUT THE REPORT PERIOD FOR REFUELING AND MAINTENANCE.

LEGEND

[1]Type:
F–Forced
S–Scheduled

[2]Reason:
A–Equipment Failure
B–Maintenance or Test
C–Refueling
D–Regulatory Restriction
E–Operator Training & License Examination
F–Administrative
G–Operational Error
H–Other

[3]Method:
1–Manual
2–Manual Scram
3–Automatic Scram
4–Continuations
5–Load Reduction
9–Other

[4]Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5]Exhibit H:
Same source.

CRYSTAL RIVER 3

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Florida/Citrus 7 miles NW of Crystal River
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2452
5.	Design Electrical Rating (MWE Net)	825
6.	Date of Initial Criticality	January 14, 1977
7.	Date of Commercial Operation	March 13, 1977
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Gulf of Mexico

Utility & Contractor Information

10.	Licensee	Florida Power Corporation
11.	Corporate Address	3201 34th Street, South St. Petersbury, Florida 33733
12.	Corporate Contact	J. Hancock
13.	Architect/Engineer	Gilbert Associates, Inc.
14.	Nuclear Steam Supply System	B&W
15.	Constructor	J. A. Jones Construction Co.

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. Lewis
18.	IE Section Chief	R. Martin
19.	IE Resident Inspector	T. F. Stetka
20.	Licensing Project Manager	P. B. Erickson
21.	Docket Number	50-302
22.	License Number/Date of Issue	DPR-72, 12/3/76
23.	Public Document Room	Crystal River Public Library 668 N.W. First Crystal River, Florida 32639

INSPECTION STATUS

ENFORCEMENT STATUS

AS REQUIRED BY FLORIDA POWER CORPORATION'S APPROVED QUALITY ASSURANCE PROGRAM, HOUSEKEEPING REQUIREMENTS WERE ESTABLISHED IN ACCORDANCE WITH REGULATORY GUIDE 1.39-1976 AND ANSI N45.2.3-1973. IMPLEMENTING INSTRUCTIONS PERTAINING TO CRYSTAL RIVER 3 ARE CONTAINED IN ADMINISTRATIVE INSTRUCTION AI-1000. ON DECEMBER 12, 1979, IT WAS OBSERVED THAT ESTABLISHED HOUSEKEEPING REQUIREMENTS HAD NOT BEEN ADHERED TO, IN THAT, EXCESSIVE AMOUNTS OF PLASTIC, CARDBOARD, WOOD, PIPING AND FITTINGS, AND GENERAL DEBRIS HAD BEEN ALLOWED TO ACCUMULATE WITHIN THE AUXILIARY BUILDING CREATING A POTENTIAL FIRE AND INDUSTRIAL SAFETY HAZARD.
(7955 INFRACTION)

AS REQUIRED BY TECHNICAL SPECIFICATION 6.8.3 TEMPORARY CHANGES TO PROCEDURES MAY BE MADE PROVIDED THE INTENT OF THE ORIGINAL PROCEDURE IS NOT ALTERED. ON MARCH 26, 1980, A TEMPORARY CHANGE TO OP 414 LEV. 5 "NITROGEN AND HYDROGEN SYSTEM" WAS USED TO SUPPLY HELIUM TO "B" STEAM GENERATOR WHICH WAS NOT IN THE ORIGINAL PROCEDURE.
(8013 DEFICIENCY)

CONTRARY TO THE REQUIREMENTS OF TECHNICAL SPECIFICATION 6.8.1.E, AS IMPLEMENTED BY EMERGENCY PROCEDURES EM-206 AND EM-207, THE LICENSEE DID NOT: (1) NOTIFY THE STATE OF FLORIDA OF A CLASS B EMERGENCY VIA NAWAS, AND (2) DID NOT MAINTAIN THE PHONE NUMBER OF THE PLANT MANAGER IN THE CONTROL ROOM COPY OF EM-206.
(8014 INFRACTION)

AS REQUIRED BY TECHNICAL SPECIFICATION 6.8.1, PROCEDURES REFERENCED IN REG. GUIDE 1.33 (1972), APPENDIX A, MUST BE FOLLOWED. SECTION G.5.D OF REG. GUIDE 1.33 (1972), APPENDIX A, LISTS PROTECTIVE CLOTHING AS AN AREA WHERE PROCEDURES ARE NEEDED. FACILITY RADIATION PROTECTION PROCEDURE RP-101, RADIATION PROTECTION MANUAL INCLUDES PROCEDURES FOR USE AND REUSE OF PROTECTIVE CLOTHING. SECTION 4.9 OF RP-101 STATES LAUNDERED PROTECTIVE CLOTHING IS ACCEPTABLE FOR REUSE IF SURVEY READINGS INDICATE A RADIATION LEVEL BELOW 1500 COUNTS PER MINUTE AS MEASURED WITH A SIDE WINDOW G-M TUBE. ON MARCH 13, 1980, SEVERAL DOZEN SETS OF LAUNDERED COVERALLS WERE FOUND WITH RADIOACTIVITY IN EXCESS OF THE ABOVE LIMITS. MEASUREMENTS INDICATED RADIATION LEVELS AS HIGH AS 7 MREM/HR, CORRESPONDING TO AN ESTIMATED 7000 TO 10000 COUNTS PER MINUTE.
(8015 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

REFUELING SHUTDOWN.

+ LAST IE SITE INSPECTION DATE: APRIL 28 - MAY 2, 1980

+ INSPECTION REPORT NO: 50-302/80-22

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
80-09/ 03L-0	02/09/80	03/10/80	30 DAY LER	DOSE EQUIVALENT I-131 EXCEEDS LIMITS.
80-10/ 01T-0	02/27/80	03/10/80	14 DAY LER	DOSE EQUIVALENT I-131 MEASUREMENT FREQUENCY NOT INCREASED AS REQUIRED BY TECHNICAL SPECIFICATIONS.
80-11/ 04L-0	02/22/80	03/18/80	ENVIRONMENTAL	FEBRUARY SAMPLE MISSED FROM TEST WELL NO. 4
80-12/ 04L-0	02/29/80	03/20/80	ENVIRONMENTAL	ENVIRONMENTAL SAMPLING NOT CONDUCTED AS REQUIRED.
80-13/ 03L-0	03/04/80	03/24/80	ENVIRONMENTAL	EXCESSIVE METABOLIC FUNCTION DEVIATION.
80-14/ 03L-0	03/13/80	03/18/80	ENVIRONMENTAL	EXCESSIVE RADIOACTIVITY IN AN ENVIRONMENTAL SAMPLE.
80-15/ 03L-0	03/06/80	04/03/80	30 DAY LER	DURING ENGINEERED SAFEGUARDS FUNCTIONAL TEST DECAY HEAT SUCTION ISOL VALVE BEGAN TO SHUT, TRIPPING DHR PUMP BOTH RCS LOOPS INOP CONTRARY TO TS 3.4.1 FOR 8 MIN.

SUMMARY

INSPECTION SUMMARY

INSPECTION JANUARY 7-10 (80-01): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 64 INSPECTOR HOURS ON SITE IN THE AREAS OF REVIEW OF OPERATIONS, IE BULLETIN FOLLOWUP AND A TOUR OF THE FACILITY AND CORPORATE HEADQUARTERS. OF THE THREE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION MARCH 17-21 (80-12): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 40 RESIDENT INSPECTOR HOURS ON SITE IN THE AREAS OF OPERATIONS, PLANT TOURS AND OBSERVATION OF WORK. OF THE THREE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION MARCH 21-28 (80-13): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 44 INSPECTOR-HOURS ON SITE IN THE AREAS OF FUEL RECEIPT AND INSPECTION AND OUTAGE MAINTENANCE. OF THE TWO AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN ONE AREA; ONE ITEM OF NONCOMPLIANCE WAS FOUND IN ONE AREA (DEFICIENCY, PROCESSING AN IMPROPER PROCEDURE CHANGE).

+ INSPECTION MARCH 18-21 (80-15): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 39 INSPECTOR-HOURS ON SITE IN THE AREAS OF RADIATION PROTECTION INCLUDING TRAINING, OUTAGE PREPARATIONS EXPOSURE CONTROL, PROCEDURES, RESPIRATORY PROTECTION, POSTING AND LABELING, AND PREVIOUSLY IDENTIFIED ITEMS. OF THE SEVEN AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN SIX AREAS; ONE ITEM OF NONCOMPLIANCE WAS FOUND IN ONE AREA (INFRACTION - CONTAMINATION OF PROTECTIVE CLOTHING, READY FOR USE, IN EXCESS OF PROCEDURAL LIMITS).

INSPECTION MARCH 10-14 (80-16): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 32 INSPECTOR-HOURS ON SITE IN THE AREAS OF NON NUCLEAR INSTRUMENTATION SYSTEM COMPONENT ALIGNMENT, AND PROCEDURES. OF THE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

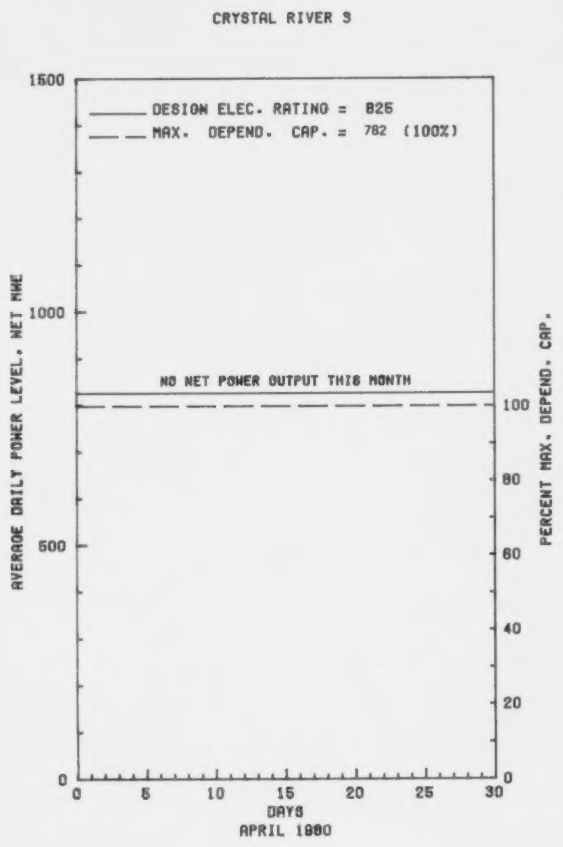
INSPECTION MARCH 19-21 AND 25-28 (80-17): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 43 INSPECTOR-HOURS ON SITE IN THE AREAS OF INSERVICE INSPECTION (ISI) - PROGRAM REVIEW, PROCEDURE REVIEW, WORK OBSERVATION AND DATA EVALUATION; IE BULLETIN 79-17; UT INSPECTION OF STEAM TURBINE DISCS; ONCE THRU STEAM GENERATOR (OSTG) TUBE TO TUBE SHEET WELD REPAIR PROGRAM. NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

REVIEWED BY OMPA LOEB: P.S. Bobe **DATE:** 6/12/80

REVIEWED BY NRR DOR: Peter B Erickson **DATE:** 6/13/80

CRYSTAL RIVER 3

AVERAGE POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-302
2. Utility Contact: M.W. CULVER (904) 795-6486
3. Licensed Thermal Power (MWt): 2452
4. Nameplate Rating (Gross MWe): 989 x 0.9 = 890
5. Design Electrical Rating (Net MWe): 825
6. Maximum Dependable Capacity (Gross MWe): 821
7. Maximum Dependable Capacity (Net MWe): 782
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	719.0	2,903.0	27,479.0
12. Hours Reactor Was Critical	.0	1,305.5	16,455.7
13. Reactor Reserve Shutdown Hours	.0	.0	1,116.9
14. Hours Generator On-Line	.0	1,297.4	15,997.7
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	3,085,927	35,402,838
17. Gross Electrical Energy (MWH)	0	1,058,396	12,030,164
18. Net Electrical Energy (MWH)	0	1,009,335	11,400,975
19. Unit Service Factor	.0	44.7	58.2
20. Unit Availability Factor	.0	44.7	58.2
*21. Unit Capacity Factor (MDC Net)	.0	44.5	53.1
22. Unit Capacity Factor (DER Net)	.0	42.1	50.3
23. Unit Forced Outage Rate	.0	18.0	32.0

24. Shutdowns Scheduled Over Next 6 Months (Type, Data, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 06/10/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-9	03/07/80	S	719.0	C	4		ZZ	ZZZZZZ	CONTINUATION OF THE REFUELING OUTAGE BEGUN ON 03/07/80.

SUMMARY

THE UNIT REMAINED SHUTDOWN THROUGHOUT THE REPORT PERIOD FOR REFUELING.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

DAVIS-BESSE 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Ohio/Ottawa 21 miles E of Toledo
2.	Electric Reliability Council	East Central Area Reliability Coordination Agreement
3.	Type of Reactor	PWR
4.	Powar Level (MWT)	2772
5.	Design Electrical Rating (MWE Net)	906
6.	Date of Initial Criticality	August 12, 1977
7.	Date of Commercial Operation	11/21/77 (25%); 12/19/77 (40%) 01/23/78 (75%); 07/31/78 (100%)
8.	Condenser Cooling Method	Cooling Tower
9.	Condenser Cooling Water Source	Lake Erie

Utility & Contractor Information

10.	Licensee	Toledo Edison Co.
11.	Corporate Address	Edison Plaza, 300 Madison Avenue Toledo, Ohio 43652
12.	Corporate Contact	R. P. Crouse, V. P., Nuclear (419)259-5221
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	B&W
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	T. N. TAMBLING
19.	IE Resident Inspector	L. Reyes (419) 898-2775
20.	Licensing Project Manager	D. Garner
21.	Docket Number	50-346
22.	License Number/Date of Issue	NPF-3, 4/22/77
23.	Public Document Room	Ida Rupp Public Library 310 Madison Street Port Clinton, OH 43452

INSPECTION STATUS

ENFORCEMENT STATUS

FAILURE TO ADEQUATELY FOLLOW LOGGING PROCEDURES AS REQUIRED IN SECURITY PLAN.
(8002 DEFICIENCY)

FAILURE TO CONDUCT SEARCHES AS REQUIRED BY SECURITY PLAN.
(8002 DEFICIENCY)

FAILURE TO ADEQUATELY AND EFFECTIVELY CONTROL ACCESS IN ACCORDANCE WITH SECURITY PLAN REQUIREMENTS.
(8002 INFRACTION)

FAILURE OF THE ALARM STATIONS TO HAVE ADEQUATE ALARM MONITORING CAPABILITIES.
(8002 INFRACTION)

AN INSPECTOR WAS ABLE TO CIRCUMVENT INTRUSION DETECTION SENSORS IN CERTAIN AREAS.
(8002 DEFICIENCY)

FAILURE TO PROVIDE AN ALARM SUBSYSTEM IN ACCORDANCE WITH A SECURITY PLAN COMMITMENT.
(8002 DEFICIENCY)

A PORTION OF THE ACTIVE INTRUSION DETECTION ALARM SYSTEM IS NOT TAMPER-INDICATING.
(8002 INFRACTION)

SEVERAL REDUNDANT SECURITY COMMUNICATION SYSTEMS WERE NOT PROPERLY SECURED.
(8002 DEFICIENCY)

THE LICENSEE FAILED TO CONDUCT PROPER TESTS OF SPECIAL PURPOSE DETECTORS.
(8002 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

FACILITY ITEMS

+ Completion in December 1979 of the increased fuel pool storage capacity from 260 fuel assembly positions to 735 positions. (80-08)

MANAGERIAL ITEMS

+ As a result of the February 29, 1980 Management Meeting, an Order to Modify the Licensee was issued March 5, 1980. The order required the licensee to take immediate and long term action to increase the qualifications of non-licensed equipment operation assigned to the operating shifts. (80-07)

PLANT STATUS

+ The unit tripped on April 7, 1980 due to partial loss of condenser vacuum. On April 8 the unit was taken to cold shutdown in preparation for the refueling outage scheduled to start April 11. On April 19 while in the refueling mode, decay heat removal was lost for approximately 2 1/2 hours due to inadvertent tripping of a 13.8KV bus breaker. Fuel movement is scheduled to start early part of May. The refueling and maintenance outage is currently scheduled to be 12 weeks long.

+LAST IE SITE INSPECTION DATE: March 5-24, 1980

+ INSPECTION REPORT NO: 80-09

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-19/ 01T-0	03/25/80	04/03/80	Action statement deadline passed before RPS could be verified.
80-20/ 03L-0	03/06/80	04/03/80	No control room indication for containment vacuum relief isolation valve.
80-21/ 03L-0	03/13/80	04/10/80	Diesel fire protection pump control problems.
80-22/ 03L-0	03/17/80	04/15/80	Control room position indication for containment analyzer discharge line valve lost.

SUMMARY

INSPECTION SUMMARY

+ Inspection on February 14-15 and March 13, (80-03): Spent Fuel Rack QA/QC programs and implementing welding and nondestructive examinations (NDE) procedures and record review; IE Bulletin 79-13 and IE Bulletin 79-17 activities. The inspection involved 20 inspector hours onsite by one NRC inspector. No items of noncompliance or deviations were identified.

+ Inspection on March 10-12, (80-04): A routine announced inspection of fire protection program items from IE Inspection Report 50-346/79-15. The inspection involved 16 inspector hours onsite by one NRC inspector. No items of noncompliance or deviations were identified.

+ Inspection on February 4-8, 11, 15, 19-22, 25-28 and March 3-7, (80-05): Routine unannounced inspection of followup on previous inspection findings, plant operations, physical protection (security organization, physical barriers, access control, communications) followup of licensee event reports, reactor trip on February 5, 1980, the receipt of new fuel and TMI-2 lesson learned followup. The inspection involved 147.5 inspector hours onsite by two NRC inspectors. No items of noncompliance or deviations were identified.

+ Meeting February 29, (80-07): An announced meeting was held with corporate and site management to discuss current operating shift staffing concerns and results of a security inspection. The meeting lasted approximately 3 1/2 hours and NRC representatives included IE Headquarters and Region III personnel. The licensee committed to immediate implementation of three short term actions to strengthen the experience level of non-licensed personnel on the five operating shifts until the reactor is shutdown for the scheduled refueling outage. For the long term the licensee was committed to meet the operating shift staffing requirements of the December 1979 draft of ANS 3.1 Standard prior to returning the reactor to operation from the refueling outage. An Order to Modify the Licensee was issued on March 5, 1980 to confirm these commitments. No items of noncompliance or deviations were identified within the scope of this meeting.

+ Inspection on March 26-28, (80-08): Facility organization and operation measurements and controls, inventory and inventory verifications, shipping and receiving, storage and internal controls, records and reports and management of material control system. The inspection involved 36 inspector hours onsite by two NRC inspectors and was begun during the regular hours. The licensee was found to be in compliance with NRC requirements in the seven areas examined during the inspections except for the following one item (failure to perform an independent fuel management audit for calendar year 1978).

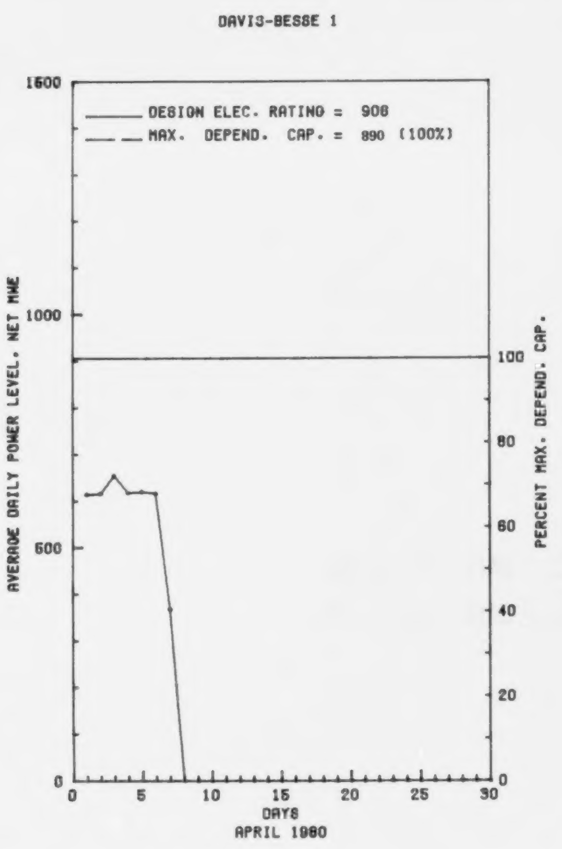
+ Inspection on March 5-24, (80-09): A special announced inspection to verify the licensee compliance with Item 1 a, b, and c of an Order to Modify the Licensee issued March 5, 1980 to upgrade the experience level of non-licensed members of the operating shifts. The inspection involved 30 inspector hours onsite by the resident inspector. No items of noncompliance or deviations were identified.

REVIEWED BY OMPA LOEB: ______ **DATE:** 6/12/80

REVIEWED BY NRR DOR: D J Garner **DATE:** 6/13/80

DAVIS-BESSE 1

AVERAGE POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-346
2. Utility Contact: BILAL SARSOUR (419) 259-5000 X236
3. Licensed Thermal Power (MWt): 2772
4. Nameplate Rating (Gross MWe): 1069 x 0.9 = 962
5. Design Electrical Rating (Net MWe): 906
6. Maximum Dependable Capacity (Gross MWe): 934
7. Maximum Dependable Capacity (Net MWe): 890
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	719.0	2,903.0	23,428.0
12. Hours Reactor Was Critical	159.3	2,078.2	13,042.4
13. Reactor Reserve Shutdown Hours	.0	.0	2,875.8
14. Hours Generator On-Line	159.3	2,008.7	11,883.4
15. Unit Reserve Shutdown Hours	.0	.0	1,728.2
16. Gross Thermal Energy (MWH)	315,675	4,687,304	24,886,811
17. Gross Electrical Energy (MWH)	104,487	1,583,559	8,307,070
18. Net Electrical Energy (MWH)	94,347	1,483,787	7,654,365
19. Unit Service Factor	22.2	69.2	52.0
20. Unit Availability Factor	22.2	69.2	60.2
21. Unit Capacity Factor (MDC Net)	14.7	57.4	38.6
22. Unit Capacity Factor (DER Net)	14.5	56.4	38.9
23. Unit Forced Outage Rate	.0	14.3	25.6

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 07/04/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
3	04/07/80	F	.0	B	3				THE REACTOR TRIPPED DUE TO THE LOSS OF CONDENSER VACUUM CAUSED BY UNINTENTIONAL LOSS OF CONDENSER VACUUM THRU HIGH LOAD DRAIN LINE WHEN LEAK TESTING FEEDWATER HEATER 1-6.
4	04/07/80	S	559.7	C	9				A UNIT OUTAGE WAS INITIATED TO PERFORM SCHEDULED MAINTENANCE AND REFUELING.

SUMMARY

THE REACTOR TRIPPED ON 04/07/80 FROM UNINTENTIONAL LOSS OF CONDENSER VACUUM DURING FEEDWATER HEATER LEAK TESTING. THE UNIT WAS SUBSEQUENTLY SHUTDOWN FOR SCHEDULED MAINTENANCE AND REFUELING WORK.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

DRESDEN 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Illinois/Grundy 9 miles E of Morris
2.	Electric Reliability Council	Mid-America Interpool Network
3.	Type of Reactor	BWR
4.	Power Level (MWT)	700
5.	Design Electrical Rating (MWE Net)	200
6.	Date of Initial Criticality	October 15, 1959
7.	Date of Commercial Operation	July 4, 1960
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Kankakee River

Utility & Contractor Information

10.	Licensee	Commonwealth Edison Co.
11.	Corporate Address	P. O. Box 767 Chicago, Illinois 60690
12.	Corporate Contact	C. Reed
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	GE
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	R. Spessard
19.	IE Resident Inspector	J. L. Barker (815)-942-9267
20.	Licensing Project Manager	Paul W. O'Connor
21.	Docket Number	50-010
22.	License Number/Date of Issue	DPR-2 issued 9/28/59
23.	Public Document Room	Morris Public Library 604 Liberty Street Morris, Illinois 60451

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

MANAGERIAL ITEMS

+ Mr. Byron Lee, Jr. was promoted to Executive Vice President and Mr. Cordell Reed, Vice President, of Commonwealth Edison Company effective February 1, 1980. (80-04)

PLANT STATUS

Unit 1 shutdown on October 31, 1978 for an extended outage to upgrade ECCS, chemical cleaning and refueling. Outage expected to last a minimum of 32 months.

+LAST IE SITE INSPECTION DATE: March 3 through April 4, 1980

+ INSPECTION REPORT NO: 80-07

SUMMARY

INSPECTION SUMMARY

+ Inspection on February 20-22 and 26, (80-04): Routine, announced inspection of Emergency Planning including internal audits; licensee coordination with offsite support agencies; and review of emergency drills; Confirmatory Measurements, including the licensee's program for quality control of nonradiological analytical measurements; evaluation of the licensee's analytical results of plant effluent samples obtained during a previous inspection; and collection of plant effluent samples for future comparative analysis; and Environmental Protection, including review of the radiological and nonradiological environmental monitoring program for 1978. The inspection involved 42 inspector hours onsite by two NRC inspectors. Of the seven areas inspected, no apparent items of noncompliance or deviations were identified in six areas; two apparent items of noncompliance were identified in one area (failure to collect and analyze fish for gamma isotopic for the first half of 1978; and failure to report analytical results of fish collected during the second half of 1978 in the Annual Report for 1978).

+ Inspection on February 27-28, (80-05): Licensee actions relative to IE Bulletin 79-14 including general discussion on NRC requirements, work procedure review and review of analysis and calculations. The inspection involved 12 inspector hours onsite by one NRC inspector. No items of noncompliance or deviations were identified.

+ Inspection on February 4-29, (80-06): Routine, unannounced resident inspection of plant operations; physical protection-security organization, physical barriers, access control (identification, authorization, badging, search, and escorting), and communications; and refueling activities. The inspection involved 138 inspector hours onsite by two NRC inspectors. No items of noncompliance were identified.

+ Inspection on March 3 through April 4, (80-07): Routine, unannounced resident inspection consisting of monthly surveillance observation, operational safety verification, and monthly maintenance observation. The inspection involved 123 inspector hours onsite by two NRC inspectors. Of the three areas inspected, there were no items of noncompliance identified.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-01/03L-0	03/03/80	03/25/80	Unit 1 fire pump was increased in capacity to be comparable to 2/3 Fire Pump for the new demands of fire system.

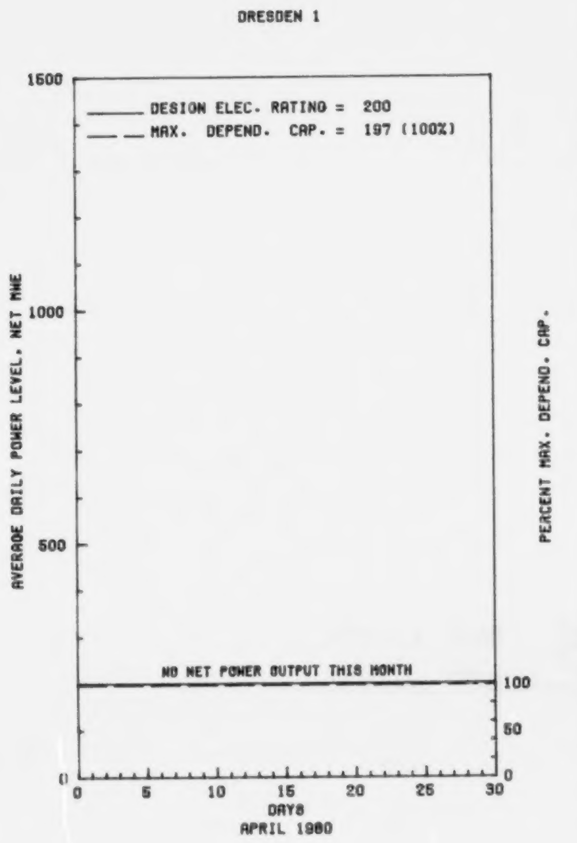
==

REVIEWED BY OMPA LOEB: Earl [illegible] **DATE:** 6-12-80

REVIEWED BY NRR DOR: Paul W O'Connor **DATE:** 6/13/80

DRESDEN 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-010
2. Utility Contact: J. A. Cieszkiewicz (815) 942-2920
3. Licensed Thermal Power (MWt): 700
4. Nameplate Rating (Gross MWe): 245.5 x 0.85 = 209
5. Design Electrical Rating (Net MWe): 200
6. Maximum Dependable Capacity (Gross MWe): 205
7. Maximum Dependable Capacity (Net MWe): 197
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	719.0	2,903.0	175,703.0
12. Hours Reactor Was Critical	.0	.0	113,153.9
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	.0	.0	111,196.2
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	0	56,014,747
17. Gross Electrical Energy (MWH)	0	0	16,799,254
18. Net Electrical Energy (MWH)	-983	-3,515	15,806,512
19. Unit Service Factor	.0	.0	63.3
20. Unit Availability Factor	.0	.0	63.3
21. Unit Capacity Factor (MDC Net)	.0	.0	45.7
22. Unit Capacity Factor (DER Net)	.0	.0	45.0
23. Unit Forced Outage Rate	.0	.0	11.2

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: July 1981

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
8	10/31/78	S	719.0	D	4				INSTALL ECCS AND UPGRADE RPS TO SATISFY COMMISSION ORDER.

SUMMARY

THE UNIT REMAINED SHUTDOWN THROUGHOUT THE REPORT PERIOD FOR REFUELING AND CHEMICAL CLEANING.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

DRESDEN 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Illinois/Grundy 9 miles E of Morris
2.	Electric Reliability Council	Mid-America Interpool Network
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2527
5.	Design Electrical Rating (MWE Net)	794
6.	Date of Initial Criticality	January 7, 1970
7.	Date of Commercial Operation	JULY, 1970
8.	Condenser Cooling Method	Cooling Lake
9.	Condenser Cooling Water Source	Kankakee and Des Plaines Rivers

Utility & Contractor Information

10.	Licensee	Commonwealth Edison Co.
11.	Corporate Address	P. O. Box 767 Chicago, Illinois 60690
12.	Corporate Contact	C. Reed
13.	Architect/Engineer	Sargent & Lundy
14.	Nuclear Steam Supply System	GE
15.	Constructor	United Engineers & Constructors, Inc.

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	R. Spessard
19.	IE Resident Inspector	J. L. Barker (815)-942-9267
20.	Licensing Project Manager	Paul W. O'Connor
21.	Docket Number	50-237
22.	License Number/Date of Issue	DPR-19 issued 12/22/69
23.	Public Document Room	Morris Public Library 604 Liberty Street Morris, Illinois 60451

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

MANAGERIAL ITEMS

+ Mr. Byron Lee, Jr. was promoted to Executive Vice President and Mr. Cordell Reed, Vice President, of Commonwealth Edison Company effective February 1, 1980. (80-04)

PLANT STATUS

Unit operating routinely.

+LAST IE SITE INSPECTION DATE: March 3 through April 4, 1980

+ INSPECTION REPORT NO: 80-07

SUMMARY

INSPECTION SUMMARY

+ Inspection on February 20-22 and 26, (80-04): Routine, announced inspection of Emergency Planning including internal audits; licensee coordination with offsite support agencies; and review of emergency drills; Confirmatory Measurements, including the licensee's program for quality control of nonradiological analytical measurements; evaluation of the licensee's analytical results of plant effluent samples obtained during a previous inspection; and collection of plant effluent samples for future comparative analysis; and Environmental Protection, including review of the radiological and nonradiological environmental monitoring program for 1978. The inspection involved 42 inspector hours onsite by two NRC inspectors. Of the seven areas inspected, no apparent items of noncompliance or deviations were identified in six areas; two apparent items of noncompliance were identified in one area (failure to collect and analyze fish for gamma isotopic for the first half of 1978; and failure to report analytical results of fish collected during the second half of 1978 in the Annual Report for 1978).

+ Inspection on February 27-28, (80-05): Licensee actions relative to IE Bulletin 79-14 including general discussion on NRC requirements, work procedure review and review of analysis and calculations. The inspection involved 12 inspector hours onsite by one NRC inspector. No items of noncompliance or deviations were identified.

+ Inspection on February 4-29, (80-06): Routine, unannounced resident inspection cf plant operations: physical protection-security organization, physical barriers, access control (identification, authorization, badging, search, and escorting), and communications; and refueling activities. The inspection involved 138 inspector hours onsite by two NRC inspectors. No items of noncompliance were identified.

+ Inspection on March 3 through April 4, (80-07): Routine, unannounced resident inspection consisting of monthly surveillance observation, operational safety verification, and monthly maintenance observation. The inspection involved 123 inspector hours onsite by two NRC inspectors. Of the three areas inspected, there were no items of noncompliance identified in two areas. There was one item of noncompliance (failure to follow radiation protection procedures) identified in one area.

REPORTS RECEIVED FROM LICENSEE

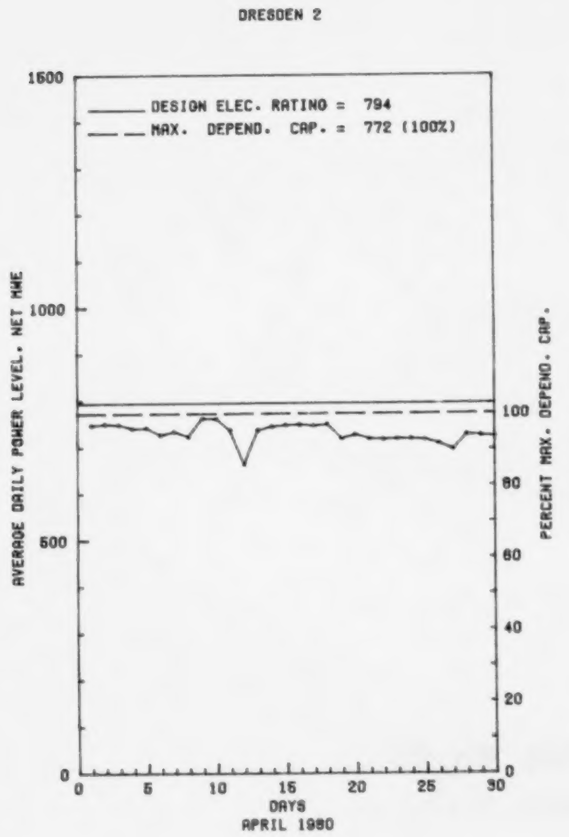
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-11/ 03L-0	03/18/80	04/03/80	DPIS 1349A tripped at 32.2 inches water increasing dp.

REVIEWED BY OMPA LOEB: Earl Hill DATE: 6-12-80

REVIEWED BY NRR DOR: Paul W O'Connor DATE: 6/13/80

DRESDEN 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-237
2. Utility Contact: S. A. Cieszkiewicz (815) 942-2920 x263
3. Licensed Thermal Power (MWt): 2527
4. Nameplate Rating (Gross MWe): 920 x 0.9 = 828
5. Design Electrical Rating (Net MWe): 794
6. Maximum Dependable Capacity (Gross MWe): 812
7. Maximum Dependable Capacity (Net MWe): 772
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	719.0	2,903.0	87,359.0
12. Hours Reactor Was Critical	719.0	2,887.7	67,424.4
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	719.0	2,878.0	64,140.7
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,742,587	6,826,370	128,468,282
17. Gross Electrical Energy (MWH)	559,014	2,180,499	41,182,120
18. Net Electrical Energy (MWH)	522,199	2,073,768	38,961,888
19. Unit Service Factor	100.0	99.1	73.4
20. Unit Availability Factor	100.0	99.1	73.4
21. Unit Capacity Factor (MDC Net)	94.1	92.5	57.8
22. Unit Capacity Factor (DER Net)	91.5	90.0	56.2
23. Unit Forced Outage Rate	.0	.9	12.9

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): Recirc. Pump Seal Repair, 5/14/80 (4 Days)
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	NONE								

SUMMARY

OPERATION WAS ROUTINE THROUGHOUT THE REPORT PERIOD AT AN AVERAGE LOAD OF 730 MWE, WITH NO OUTAGES OR FURTHER REDUCTIONS REPORTED.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

DRESDEN 3

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Illinois/Grundy 9 miles E of Morris
2.	Electric Reliability Council	Mid-America Interpool Network
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2527
5.	Design Electrical Rating (MWE Net)	794
6.	Date of Initial Criticality	January 31, 1971
7.	Date of Commercial Operation	November 16, 1971
8.	Condenser Cooling Method	Cooling Lake
9.	Condenser Cooling Water Source	Kankakee and Des Plaines Rivers

Utility & Contractor Information

10.	Licensee	Commonwealth Edison Co.
11.	Corporate Address	P. O. Box 767 Chicago, Illinois 60690
12.	Corporate Contact	D. L. Peoples
13.	Architect/Engineer	Sargent & Lundy
14.	Nuclear Steam Supply System	GE
15.	Constructor	United Engineers & Constructors, Inc.

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	R. Spessard
19.	IE Resident Inspector	J. L. Barker 815-942-9267
20.	Licensing Project Manager	R. Bevan
21.	Docket Number	50-249
22.	License Number/Date of Issue	DPR-25 issued 1/12/71
23.	Public Document Room	Morris Public Library 604 Liberty Street Morris, Illinois 60451

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS

- Unit 3 jet pump held down beams 2, 5, 9, 11, 13 and 17 were replaced with forged inconel X-750 material. Jet pump beam 13 failed during operation, and beams 2, 5, 9, 11 and 17 were replaced after crack indications were identified during insitu ultrasonic testing. (80-10)

MANAGERIAL ITEMS

- Mr. Byron Lee, Jr. was promoted to Executive Vice President and Mr. Cordell Reed, Vice President, of Commonwealth Edison Company effective February 1, 1980. (80-04)

PLANT STATUS

- The unit began startup on April 24, 1980 following completion of the refueling/maintenance outage that began on February 2, 1980, however, return of the unit to service has been delayed due to problems experienced with some of the ADS valves.

*LAST IE SITE INSPECTION DATE: March 3 through April 4, 1980

- INSPECTION REPORT NO: 80-10

SUMMARY

INSPECTION SUMMARY

- Inspection on February 20-22 and 26, (80-04): Routine, announced inspection of Emergency Planning including internal audits; licensee coordination with offsite support agencies; and review of emergency drills; Confirmatory Measurements, including the licensee's program for quality control of nonradiological analytical measurements; evaluation of the licensee's analytical results of plant effluent samples obtained during a previous inspection; and collection of plant effluent samples for future comparative analysis; and Environmental Protection, including review of the radiological and nonradiological environmental monitoring program for 1978. The inspection involved 42 inspector hours onsite by two NRC inspectors. Of the seven areas inspected, no apparent items of noncompliance or deviations were identified in six areas; two apparent items of noncompliance were identified in one area (failure to collect and analyze fish for gamma isotopic for the first half of 1978; and failure to report analytical results of fish collected during the second half of 1978 in the Annual Report for 1978).
- Inspection on February 27-28, (80-05): Licensee actions relative to IE Bulletin 79-14 including general discussion on NRC requirements, work procedure review and review of analysis and calculations. The inspection involved 12 inspector hours onsite by one NRC inspector. No items of noncompliance or deviations were identified.
- Inspection on February 21-22 and 27-29, (80-06): Inservice, NDE examination. Review of examination program, review of examination procedures, observation of examination activites, review of examination records. The inspection involved 48 inspector hours onsite by one NRC inspector. No items of noncompliance or deviations were identified.
- Inspection on February 4-29, (80-07): Routine, unannounced resident inspection of plant operations; physical protection-security organization, physical barriers, access control (identification, authorization, badging, search, and escorting), and communications; and refueling activities. The inspection involved 138 inspector hours onsite by two NRC inspectors. No items of noncompliance were identified.
- Inspection on March 14 and 18, (80-08): Jet pump hold down beam ultrasonic examination (UT) procedure, calibration, and record review. The inspection involved eight inspecter hours onsite by two NRC inspectors. No items of noncompliance or deviations were identified.
- Inspection on March 3 through April 4, (80-10): Routine, unannounced resident inspection consisting of monthly surveillance observation, operational safety verification, and monthly maintenance observation. The inspection involved 123 inspecter hours onsite by two NRC inspectors. Of the three areas inspected, there were no items of noncompliance identified in two areas. There was one item of noncompliance (failure to follow radiation protection procedures) identified in one area.

REPORTS RECEIVED FROM LICENSEE

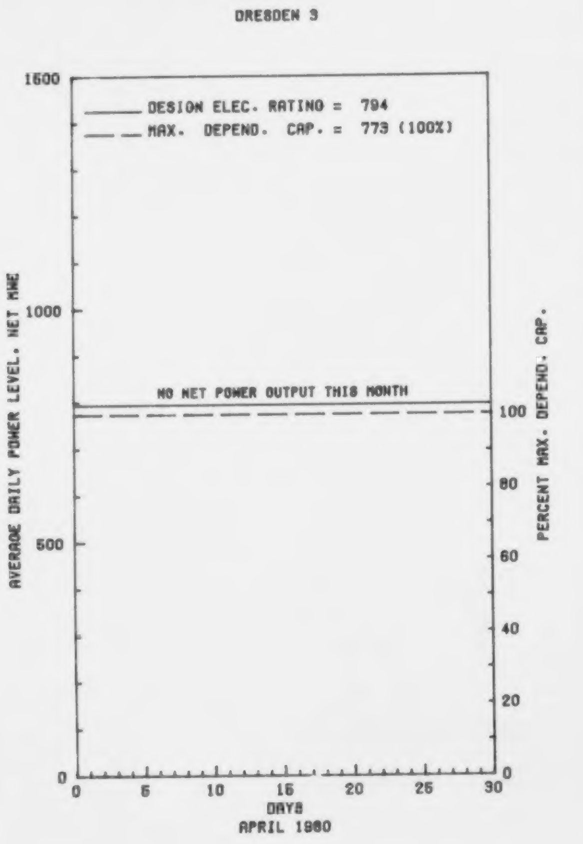
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-08/ 03L-0	03/07/80	03/25/80	Three temperature switches failed to trip during temp. switch calibration surveillance.
80-09/ 03L-0	03/06/80	03/25/80	During LPCI logic tests, injection valve MO 3-1501-22A failed to open.
80-10/ 03L-0	03/01/80	03/26/80	During SBLC relief valve setting test, valves RV 1105A and B actuated at 1525 psig and 1520 psig respectively.
80-11/ 03L-0	03/05/80	03/28/80	Switch TS 3-261-17B tripped at 203.1 deg. F.
80-12/ 03L-0	03/13/80	04/09/80	ARM Channels "A" and "B" tripped at 100 MR/hr and Rx. vent ARM Channel A tripped at 11 MR/hr.
80-15/ 01T-0	04/02/80	04/14/80	During refueling, reactor water cleanup mechanical snubber found fully retracted.

REVIEWED BY OMPA LOEB: Earl DATE: 6/12/80

REVIEWED BY NRR DOR: RBevan DATE: 6/13/80

DRESDEN 3

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-249
2. Utility Contact: S. A. Cieszkiewicz (815) 942-2920 x263
3. Licensed Thermal Power (MWt): 2527
4. Nameplate Rating (Gross MWe): 920 x 0.9 = 828
5. Design Electrical Rating (Net MWe): 794
6. Maximum Dependable Capacity (Gross MWe): 812
7. Maximum Dependable Capacity (Net MWe): 773
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	719.0	2,903.0	76,944.0
12. Hours Reactor Was Critical	.0	785.0	56,550.4
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	.0	778.7	54,103.3
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	1,327,726	103,599,394
17. Gross Electrical Energy (MWH)	14	424,685	33,854,359
18. Net Electrical Energy (MWH)	-3,924	389,935	32,083,226
19. Unit Service Factor	.0	26.8	70.3
20. Unit Availability Factor	.0	26.8	70.3
21. Unit Capacity Factor (MDC Net)	.0	17.4	53.9
22. Unit Capacity Factor (DER Net)	.0	16.9	52.5
23. Unit Forced Outage Rate	.0	2.7	16.3

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 05/03/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
2	02/03/80	S	719.0	C	4		RC	FUELXX	REFUELING OUTAGE CONTINUES.

SUMMARY

THE UNIT REMAINED SHUTDOWN THROUGHOUT THE REPORT PERIOD FOR REFUELING.

LEGEND

[1] Type:
F–Forced
S–Scheduled

[2] Reason:
A–Equipment Failure
B–Maintenance or Test
C–Refueling
D–Regulatory Restriction
E–Operator Training & License Examination
F–Administrative
G–Operational Error
H–Other

[3] Method:
1–Manual
2–Manual Scram
3–Automatic Scram
4–Continuations
5–Load Reduction
9–Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

NRC POWER RESTRICTIONS: NONE

DUANE ARNOLD

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Iowa/Linn 8 miles NW of Cedar Rapids
2.	Electric Reliability Council	Mid-Continent Area Reliability Coordination Agreement
3.	Type of Reactor	BWR
4.	Power Level (MWT)	1658
5.	Design Electrical Rating (MWE Net)	538
6.	Date of Initial Criticality	March 23, 1974
7.	Date of Commercial Operation	February 1, 1975
8.	Condenser Cooling Method	Cooling Towers
9.	Condenser Cooling Water Source	Cedar Rapids River Circulating Water System

Utility & Contractor Information

10.	Licensee	Iowa Electric Light and Power
11.	Corporate Address	IE Towers, P. O. Box 351 Cedar Rapids, Iowa 52406
12.	Corporate Contact	D. Arnold, President
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	GE
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	W. S. LITTLE
19.	IE Resident Inspector	G. C. WRIGHT/W. F. CHRISTIANSON
20.	Licensing Project Manager	T. Kevern
21.	Docket Number	50-331
22.	License Number/Date of Issue	DPR-49 issued 2/22/74
23.	Public Document Room	Reference Service Cedar Rapids Public Library 428 Third Avenue, S.E. Cedar Rapids, Iowa 52401

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

FACILITY ITEMS

- The licensee has purchased a chair-type whole body counter for routine use. The rental supine-type counter has been returned to the contractor. (80-01)

PLANT STATUS

- The unit was returned to service on April 17, 1980 following the scheduled refueling outage.

+LAST IE SITE INSPECTION DATE: March 18-19, 1980

- INSPECTION REPORT NO: 80-05

SUMMARY

INSPECTION SUMMARY

- Inspection on February 20-22, (80-01): Routine, unannounced inspection of operational radiation protection program during refueling and major main- tenance, including: qualifications; training; planning and preparation; exposure control; posting, labeling, and control; surveys; and ALARA. The inspection involved 48 inspector hours onsite by two NRC inspectors. No items of noncompliance or deviations were identified.

- Inspection on February 12-19 and March 5-7, (80-02): Review of ISI program, procedures, personnel and equipment certifications, observation of work activities, and review of NDE examination documentation. The inspection involved 72 inspector hours onsite by one NRC inspector. No items of noncompliance or deviations were identified.

- Inspection on February 19-22, (80-03): Routine, unannounced inspection of: Pre-refueling activities; Refueling activities; General Employee Training; Requalification Training; and LER followup. The inspection involved 53 inspector hours (including 4 hours off-shift) onsite by two NRC inspectors. Of the areas inspected, three items of noncompliance (failure to conduct fire brigade training; inadequate procedure; and failure to conduct monthly fire extinguisher inspections) were identified.

- Inspection on March 18-21, (80-04): Routine, unannounced inspection of refueling maintenance activities. Also follow-up on unplanned release of radioactive material from the site. The inspection involved 29 inspector hours onsite by two NRC inspectors and four inspector hours at the corporate office by one NRC inspector. Of the areas inspected, two items of noncompliance (failure to review and approve temporary procedure changes; and failure to take prompt corrective action on items identified during QC inspections) were identified.

- Inspection on March 18-19, (80-05): Nonroutine, announced special inspection of circumstances concerning an unplanned release of radioactive effluent to an unrestricted area. The inspection involved 21 inspector hours onsite by two NRC inspectors. Two items of noncompliance were identified: 10 CFR 50.72, failure to notify NRC within required time after an unplanned release; Technical Specification 3.3.1.B.2, failure to evaluate sample effluent prior to release.

REPORTS RECEIVED FROM LICENSEE

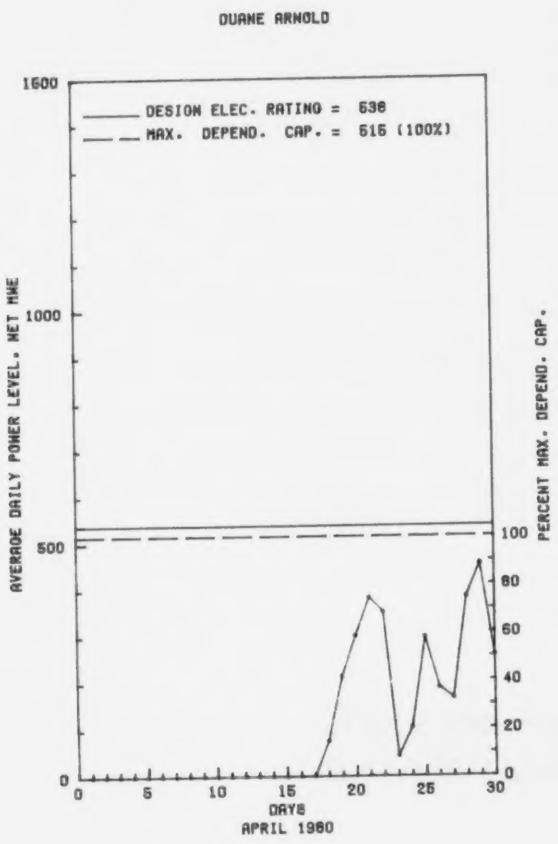
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-10/ 01T-0	03/10/80	03/24/80	Three relief valves were found to have out of specification as-found setpoints.
80-11/ 01T-0	03/17/80	03/31/80	Lower crankshaft main thrust bearing was wiped on both the thrust and journal surfaces, for Generator IG-21.
80-12/ 01T-0	03/17/80	03/31/80	Lower crankshaft thrust No. 13 bearing was found wiped on the journal surface for Generator IG-31.
80-13/ 01T-0	03/30/80	04/10/80	Fuel assembly was loaded into cell 06-23 without control rod 06-23 being fully inserted.
80-14/ 01T-0	04/02/80	04/10/80	Electrician incorrectly opened breaker between battery charger and 125 VDC bus ID-20 after battery was disconnected.

REVIEWED BY OMPA LOEB: Earl Hill DATE: 6/12/80

REVIEWED BY NRR DOR: Ken Eccleston DATE: 6/13/80

DUANE ARNOLD

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-331
2 Utility Contact: J. Van Sickel (319) 851-5611
3. Licensed Thermal Power (MWt): 1658
4. Nameplate Rating (Gross MWe): 663 x 0.9 = 597
5. Design Electrical Rating (Net MWe): 538
6. Maximum Dependable Capacity (Gross MWe): 545
7. Maximum Dependable Capacity (Net MWe): 515
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	719.0	2,903.0	45,983.0
12. Hours Reactor Was Critical	314.1	1,258.2	31,822.3
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	267.3	1,211.4	31,030.5
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	258,024	1,477,992	38,607,552
17. Gross Electrical Energy (MWH)	84,294	512,630	12,936,996
18. Net Electrical Energy (MWH)	78,041	479,390	12,094,454
19. Unit Service Factor	37.2	41.7	67.5
20. Unit Availability Factor	37.2	41.7	67.5
21. Unit Capacity Factor (MDC Net)	21.1	32.1	51.1
22. Unit Capacity Factor (DER Net)	20.2	30.7	48.9
23. Unit Forced Outage Rate	.0	.0	20.2

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
1	02/09/80	S	408.1	C	4				CONTINUATION OF REFUELING AND MAINTENANCE OUTAGE.
2	04/23/80	S	28.0	B	2		CB	ZZZZZZ	TEST OF RECIRCULATION PUMP TRIP SYSTEM.
3	04/26/80	S	15.6	B	3		CB	ZZZZZZ	TEST OF RECIRCULATION PUMP TRIP SYSTEM.
4	04/30/80	F	.0	A	5		CB	MOTORX	FORCED POWER REDUCTION DUE TO THE FAILURE OF THE "A" RECIRCULATION SYSTEM M-G SET DRIVE MOTOR.

SUMMARY

THE UNIT CAME ON LINE ON 04/18/80 FROM A REFUELING OUTAGE. LOAD WAS REDUCED ON 04/30/80 WHEN THE "A" RECIRCULATION SYSTEM MOTOR-GENERATOR SET DRIVE MOTOR FAILED.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

FARLEY 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Alabama/Houston 24 miles SE of Dothan
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2652
5.	Design Electrical Rating (MWE Net)	829
6.	Date of Initial Criticality	August 9, 1977
7.	Date of Commercial Operation	December 1, 1977
8.	Condenser Cooling Method	Once Through With Forced Draft Cooling Towers
9.	Condenser Cooling Water Source	Chattahoochee River

Utility & Contractor Information

10.	Licensee	Alabama Power Company
11.	Corporate Address	600 North 18th Street Birmingham, Alabama
12.	Corporate Contact	F. L. Clayton, Jr. Senior Vice President (205-323-5341)
13.	Architect/Engineer	Bechtel/SSI
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Daniel

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. Lewis
18.	IE Section Chief	R. D. Martin
19.	IE Resident Inspector	W. H. Bradford (205-899-3386)
20.	Licensing Project Manager	E. A. Reeves
21.	Docket Number	50-348
22.	License Number/Date of Issue	NPF-2, 6/25/77
23.	Public Document Room	G. S. Houston Memorial Library 212 W. Berdeshaw Street Dothan, Alabama 36301

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

100% POWER OPERATION.

+ LAST IE SITE INSPECTION DATE: MARCH 31 - APRIL 25, 1980

+ INSPECTION REPORT NO: 50-348/80-10

SUMMARY

INSPECTION SUMMARY

INSPECTION MARCH 25-26 (80-4): THIS ROUTINE ANNOUNCED INSPECTION INVOLVED 14 INSPECTOR-HOURS ON SITE IN THE AREAS OF PIPE SUPPORT BASEPLATE DESIGNS USING CONCRETE EXPANSION ANCHOR BOLTS (IEB 79-02); AND SEISMIC ANALYSIS FOR AS-BUILT SAFETY RELATED PIPING SYSTEMS (IEB 79-14). OF THE TWO AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION MARCH 5-7 (80-06): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 20 INSPECTOR-HOURS ON SITE IN THE AREA OF MAINTENANCE. OF THE ONE AREA INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION FEBRUARY 4-29 (80-07): THIS ROUTINE INSPECTION INVOLVED 67 INSPECTOR-HOURS ON SITE IN THE AREAS OF PLANT OPERATIONS, AND PLANT SECURITY. OF THE TWO AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION MARCH 11-14 (80-08): THIS INSPECTION INVOLVED 20 INSPECTOR HOURS ON SITE BY ONE NRC INSPECTOR IN THE AREAS OF REVIEW OF SECURITY ORGANIZATION; LIGHTING; ACCESS CONTROL (IDENTIFICATION, AUTHORIZATION AND BADGING); DETECTION AIDS; LOCKS/KEYS; RECORDS/REPORTS; AND FOLLOWUP ON PREVIOUS ENFORCEMENT MATTERS. OF THE SIX AREAS EXAMINED DURING THE INSPECTION, ONE UNRESOLVED ITEM WAS IDENTIFIED (ACCESS CONTROL - PACKAGES HAND-CARRIED INTO THE PROTECTED AREA BY SECURITY FORCE MEMBERS WERE NOT SEARCHED AT THE ACCESS CONTROL POINT).

+ INSPECTION MARCH 28 (80-11): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 104 INSPECTOR-HOURS ON SITE IN THE AREAS OF PLANT OPERATION, AND PLANT SECURITY. OF THE TWO AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

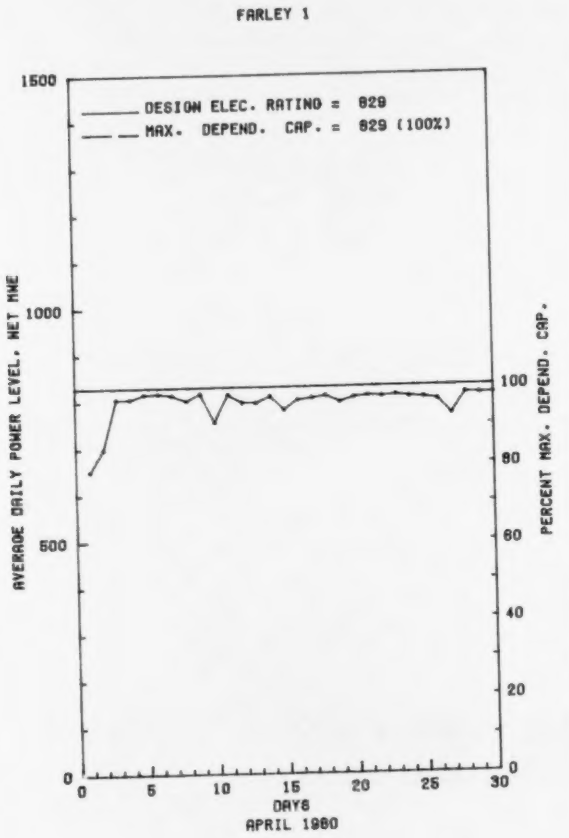
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
80-07/ 03L-0	02/02/80	02/29/80	30 DAY LER	B TRAIN SERVICE WATER DECLARED INOPERABLE WHEN PUMP WOULD NOT AUTO START DURING LOSS OF SITE POWER TEST. CAUSE WAS IMPROPER INSTALLING OF TEST EQUIPMENT.
80-09/ 03L-0	02/01/80	02/29/80	30 DAY LER	OUTPUT BREAKER FOR DIESEL GENERATOR 1B FAILED TO CLOSE AUTOMATICALLY DURING LOSS OF OFFSITE POWER TEST.CAUSE WAS A MISALIGNED SWITCH AT BREAKER PANEL.
80-10/ 03L-0	02/01/80	02/29/80	30 DAY LER	MOTOR DRIVEN AUX FEEDWATER TRAIN A INOPERABLE WHEN VALVES DID NOT OPEN ON AUTO START ON LOSS OF SITE POWER TEST.CAUSE WAS A MISPOSITIONED SWITCH AT BREAKER PANEL.
80-11/ 03L-0	02/02/80	02/29/80	30 DAY LER	CONTAINMENT ISOLATION VALVE SW FOR RCP AIR COOLER FAILED TO FULLY CLOSE DURING A SURVEILLANCE TEST, AND DECLARED INOPERABLE.CAUSE WAS THERMAL OVERLOAD SIZED MARGINALLY.
80-12/ 03L-0	02/01/80	02/29/80	30 DAY LER	COMPONENT COOLING WATER PUMP 1B FAILED TO AUTO-START DURING A LOSS OFFSITE POWER TEST, AND DECLARED INOPERABLE. CAUSE DUE PUMP 1C SWITCH MISPOSITIONED.
80-13/ 03L-0	02/02/80	02/29/80	30 DAY LER	CONTAINMENT ISOLATION VALVE (CCW TO EXCESS LETDOWN/ RCDT HEAT EXCHANGER) DID NOT FULLY CLOSE DURING TEST.
80-14/ 03L-0	02/14/80	02/29/80	14 DAY LER	FAILURE OF MOTORDRIVEN AUXFEEDWATER PUMPS TO START ON LOSS OF BOTH MAIN FEEDWATER PUMPS DUE TO INCORRECT INSTALLATION OF BYPASS SWITCHES.
80-15/ 03L-0	02/17/80	03/14/80	30 DAY LER	DEH SYSTEM FAILED CAUSING A LOAD INCREASE WHICH RESULTED IN TAVG DECREASING TO 539 DEGREES F.TAVG WAS RESTORED ABOVE 541 DEGREES F WITHIN 2 MINS,VENDOR CHECK IN PROGRESS.
80-16/ 03L-0	02/15/80	03/14/80	30 DAY LER	THE TURBINE DRIVEN AUXILIARY FEEDWATER PUMP THRUST BEARING OVERHEATED BECAUSE THE BEARING OILER WAS POSITIONED TOO HIGH. OILER ASSEMBLY REPOSITIONED – TEST SATISFACTORY.
80-17/ 03L-0	02/22/80	03/14/80	30 DAY LER	TURBINE DRIVEN AUXILIARY FEEDWATER PUMP TRIPPED ON OVERSPEED WHEN OUT OF ADJUSTMENT LIMIT-SWITCH ALLOWED GOVERNER VALVES FULL OPEN BEFORE STEAM SUPPLY–SWITCH ADJUSTED.
80-18/ 03L-0	02/29/80	03/21/80	30 DAY LER	PRESSURIZER PRESSURE CHANNEL, PT-457, INOPERABLE WHEN TRIP SETPOINT REQUIREMENTS NOT MET.DIRTY GAIN POT IN LEAD/LAG CIRCUIT. ITEM CLEANED.
80-19/ 03L-0	02/29/80	03/21/80	30 DAY LER	CONTROL ROOM EMERGENCY VENTILATION TRAIN B INOPERABLE WHEN RECIRCULATION FAN 1B WOULD NOT START BECAUSE OF A SHORTED ALARM RELAY COIL AND BLOWN FUSE - BOTH REPLACED.

REVIEWED BY OMPA LOEB: T. C. Cintula **DATE:** 6-12-80

REVIEWED BY NRR DOR: Ed A. Reeves **DATE:** 6-13-80

FARLEY 1

AVERAGE POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-348
2. Utility Contact: W. G. Hairston,III (205) 899-5156
3. Licensed Thermal Power (MWt): 2652
4. Nameplate Rating (Gross MWe): 1045 x 0.85 = 888
5. Design Electrical Rating (Net MWe): 829
6. Maximum Dependable Capacity (Gross MWe): 860
7. Maximum Dependable Capacity (Net MWe): 829
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	719.0	2,903.0	21,167.0
12. Hours Reactor Was Critical	719.0	2,466.6	13,402.1
13. Reactor Reserve Shutdown Hours	.0	436.4	1,841.7
14. Hours Generator On-Line	719.0	2,403.7	12,994.2
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,870,270	6,031,784	32,629,799
17. Gross Electrical Energy (MWH)	602,082	1,947,668	10,471,706
18. Net Electrical Energy (MWH)	571,758	1,848,102	9,907,519
19. Unit Service Factor	100.0	82.8	61.4
20. Unit Availability Factor	100.0	82.4	61.4
21. Unit Capacity Factor (MDC Net)	95.9	76.8	56.5
22. Unit Capacity Factor (DER Net)	95.9	76.8	56.5
23. Unit Forced Outage Rate	.0	5.1	7.1

24. Shutdowns Scheduled Over Next 6 Months (Type,Date,and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
032	03/31/80	F	35.0	H	5		HH	ZZZZZZ	CONTINUED POWER REDUCTION FROM 03/31/80. UNIT BEING HELD AT 700 MWE FOR SECONDARY SIDE TROUBLE-SHOOTING AND MAINTENANCE.
033	04/02/80	F	6.2	H	5		HH	ZZZZZZ	UNIT POWER INCREASED AND HELD AT 770 MWE FOR SECONDARY SIDE TROUBLE-SHOOTING AND MAINTENANCE.
034	04/10/80	F	9.6	H	5		IA	INSTRU	UNIT POWER REDUCED AND HELD AT 700 MWE TO TROUBLESHOOT NUCLEAR INSTRUMENTATION SPIKING ON CHANNEL N44. I&C CONCLUDED THAT THE SPIKING PROBABLY RESULTED FROM A MOMENTARY DIP IN INVERTER VOLTAGE; HOWEVER, THE CAUSE OF THE INVERTER DIP WAS NOT DETERMINED.
035	04/12/80	S	5.1	B	5		HA	VALVEX	UNIT WAS REDUCED TO AND HELD AT 765 MWE FOR MAIN TURBINE GOVERNOR VALVE TESTING.
036	04/15/80	F	11.3	H	5		HH	ZZZZZZ	UNIT WAS REDUCED TO AND HELD AT 770 MWE FOR SECONDARY SIDE TROUBLE-SHOOTING AND MAINTENANCE.
037	04/19/80	S	3.8	B	5		HA	VALVEX	UNIT WAS REDUCED TO AND HELD AT 770 MWE FOR MAIN TURBINE GOVERNOR VALVE TESTING.
038	04/26/80	S	2.6	B	5		HA	VALVEX	UNIT WAS REDUCED TO AND HELD AT 762 MWE FOR MAIN TURBINE GOVERNOR VALVE TESTING.

SUMMARY

OPERATION WAS ROUTINE DURING THE REPORT PERIOD, WITH SEVERAL MINOR REDUCTIONS FOR TESTING, MAINTENANCE, AND SECONDARY SIDE TROUBLE-SHOOTING.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

FITZPATRICK

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	New York/Oswego 8 miles NE of Oswego
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2436
5.	Design Electrical Rating (MWE Net)	821
6.	Date of Initial Criticality	November 17, 1974
7.	Date of Commercial Operation	July 28, 1975
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Lake Ontario

Utility & Contractor Information

10.	Licensee	Power Authority of the State of New York
11.	Corporate Address	10 Columbus Circle New York, New York 10019
12.	Corporate Contact	P. Lyon, Manager of Nuclear Operations
13.	Architect/Engineer	Stone & Webster
14.	Nuclear Steam Supply System	GE
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner (X246)
18.	IE Section Chief	H. B. KISTER
19.	IE Resident Inspector	W. H. BAUNACK
20.	Licensing Project Manager	P. Polk (X7872) 5)
21.	Docket Number	50-333
22.	License Number/Date of Issue	DPR-59, 10/17/74
23.	Public Document Room	State University College of Oswego Penfield Library - Documents Oswego, NY 13126 (315)341-2323

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

- The offgas system recombiner was out of service, resulting in higher than normal stack releases.

MANAGERIAL ITEMS:

- The position of Superintendent of Power was filled on April 18, 1980 by the Maintenance Supervisor.

PLANT STATUS:

- Shutdown for a 65-day refueling outage. The facility had been scheduled to shutdown on 5/9. However, a unit scram occurred on 5/6 due to an EHC system problem and it was decided to commence the refueling outage at that time. Representative stack and offgas release rates were 2,000 and 8,000 microcuries per second respectively.

- LAST IE SITE INSPECTION DATE: 4/25/80

- INSPECTION REPORT NO: 50-333/80-06

SUMMARY

INSPECTION SUMMARY

- 50-333/80-02 - Mar 3-14: Routine, unannounced inspection of plant operations, licensee action on SBLOCA procedure and training implementation, Licensee Event Reports (LERs), licensee action on IE Circulars, and licensee action on previous inspection findings. This report also documents inspector attendance at the TMI Lessons Learned meeting held on March 13, inspector attendance at the March 5 meeting to discuss details related to establishment of an on-site office for a full-time NRC inspector at the facility, and the in-office review of LERs, monthly operating reports, and the 10 CFR 50.59 report for 1979. The inspection began at 5:00 p.m. on Sunday, March 2, and involved 87.5 hours onsite by one region-based NRC inspector. Noncompliances: One (failure to properly search a non-regular licensee employee).

- 50-333/80-04 - Mar 31 - Apr 4: Routine, unannounced inspection of plant operations as part of a turnover of NRC Principal/Resident Inspectors, licensee progress on pipe hanger installations and modifications and licensee action on previous inspection items. This inspection also included an independent review of the instrumentation required to be environmentally qualified in accordance with IE Bulletin 79-01B. The inspection involved 91.5 hours onsite by two region-based inspectors and one resident inspector trainee and 10 hours onsite by one Reactor Projects Section Chief. Noncompliances: None.

REPORTS RECEIVED FROM LICENSEE

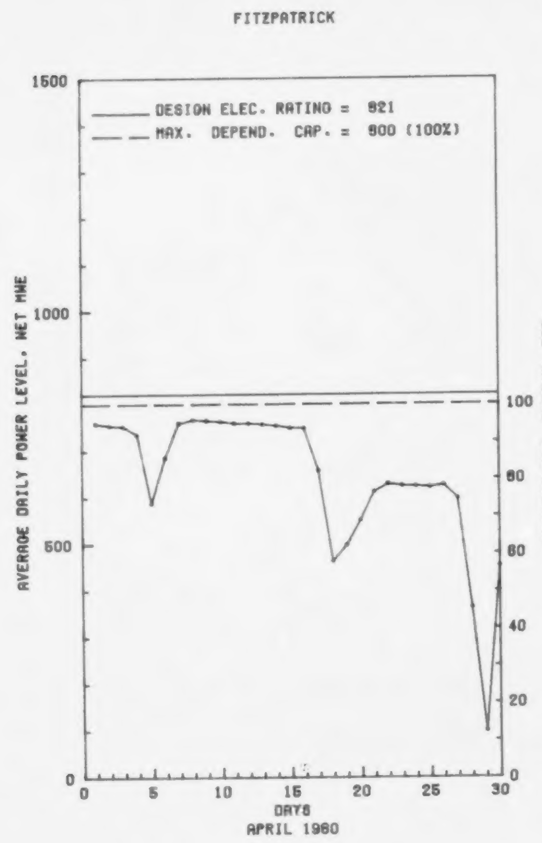
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-025/03L	02/28/80	03/28/80	APRM CH B remained inop when function switch returned to operate position
80-026/03L	03/07/80	04/03/80	During normal steady state operation a drifting LPRM caused the process computer to indicate operation in excess of thermal limits
80-027/03L	03/22/80	04/17/80	HPCI made inoperable to permit repair of a pipe support

REVIEWED BY OMPA LOEB: Earl Lell **DATE:** 6/12/80

REVIEWED BY NRR DOR: Philip J. Polk **DATE:** 6/16/80

FITZPATRICK

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

ITEM 21 CALCULATED BY USING A WEIGHTED AVERAGE

1. Docket Number: 50-333
2. Utility Contact: E. Zufelt (315) 342-3840
3. Licensed Thermal Power (MWt): 2436
4. Nameplate Rating (Gross MWe): 981 x 0.9 = 883
5. Design Electrical Rating (Net MWe): 821
6. Maximum Dependable Capacity (Gross MWe): 830
7. Maximum Dependable Capacity (Net MWe): 800
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	719.0	2,903.0	41,736.0
12. Hours Reactor Was Critical	701.0	2,793.5	29,423.7
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	697.9	2,750.4	28,488.4
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,386,216	5,631,384	56,819,578
17. Gross Electrical Energy (MWH)	478,040	1,939,570	19,293,270
18. Net Electrical Energy (MWH)	462,420	1,876,825	18,667,400
19. Unit Service Factor	97.1	94.7	68.3
20. Unit Availability Factor	97.1	94.7	68.3
*21. Unit Capacity Factor (MDC Net)	80.4	80.8	61.3
22. Unit Capacity Factor (DER Net)	78.3	78.7	54.5
23. Unit Forced Outage Rate	2.9	5.3	19.7

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
SNUBBER INSPECT & REFUEL OUTAGE - MAY 8.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
10	04/05/80	S	.0	H	5				POWER REDUCED FOR CONTROL ROD PATTERN ADJUSTMENT.
11	04/17/80	F	.0	H	5				POWER REDUCED DUE TO HIGH CONDUCTIVITY REACTOR WATER.
12	04/28/80	F	21.1	H	3				REACTOR SCRAM DUE TO GROUND BECAUSE OF CONDENSATE FORMING ON THE GENERATOR EXCITER RECTIFIERS.

SUMMARY

OPERATION WAS ROUTINE DURING THE REPORT PERIOD, WITH LOAD REDUCTIONS ON 04/05/80 FOR CONTROL ROD ADJUSTMENT AND FROM 04/17/80 THROUGH 04/20/80 TO REPAIR CONDENSER TUBE LEAKS CAUSING HIGH CONDUCTIVITY IN THE REACTOR VESSEL. ON 04/28/80, THE REACTOR SCRAMMED WHEN CONDENSATE FORMING ON THE GENERATOR EXCITER RECTIFIERS CAUSED A GROUND.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

FORT CALHOUN

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Nebraska/Washington 19 miles N of Omaha
2.	Electric Reliability Council	Mid-Continent Area Reliability Coordination Agreement
3.	Type of Reactor	PWR
4.	Power Level (MWT)	1420
5.	Design Electrical Rating (MWE Net)	457
6.	Date of Initial Criticality	August 6, 1973
7.	Date of Commercial Operation	June 20, 1974
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Missouri River

Utility & Contractor Information

10.	Licensee	W. C. Jones Division Manager – Production Operations Omaha Public Power District
11.	Corporate Address	1623 Harney Street Omaha, Nebraska 68102
12.	Corporate Contact	T. E. Short Assistant General Manager
13.	Architect/Engineer	Gibbs & Hill, Inc.
14.	Nuclear Steam Supply System	Combustion Engineering
15.	Constructor	Gibbs & Hill, Durham & Richardson, Inc.

Regulatory Information

16.	IE Region responsible	IV
17.	IE Branch Chief	G. L. Madsen
18.	IE Section Chief	T. F. Westerman
19.	IE Resident Inspector	D. Kelley
20.	Licensing Project Manager	P. C. Wagner
21.	Docket Number	50-285
22.	License Number/Date of Issue	DPR-40, 5/24/73
23.	Public Document Room	W. Dale Clark Library 215 S. 15th Street Omaha, Nebraska 68102

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO SECTION 5.8.1 OF THE LICENSEE'S TECHNICAL SPECIFICATION WHICH REQUIRES WRITTEN PROCEDURES BE ESTABLISHED AND IMPLEMENTED, THE LICENSEE CERTIFIED AN INDIVIDUAL QUALIFIED TO WEAR A RESPIRATOR WITHOUT THE MANDATORY TWO(2) YEAR RETEST.
(8003 INFRACTION)

CONTRARY TO SECTION 5.8.1 OF THE LICENSEE'S TECHNICAL SPECIFICATION. WHICH REQUIRES WRITTEN PROCEDURES BE ESTABLISHED AND IMPLEMENTED, THE LICENSEE FAILED TO USE THE CORRECT PROCEDURE TO REPAIR MAIN STEAM SAFETY VALVE MS-MV-281.
(8003 INFRACTION)

CONTRARY TO THE LICENSEE'S COMMITMENT TO REGULATORY GUIDE 8.15 AND NUREG-0041, ONE OF THE LICENSEE'S EMPLOYEE'S WAS OBSERVED WEARING A FULL FACE RESPIRATOR WHILE WEARING CONTACT LENSES.
(8003 DEVIATION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE

FACILITY ITEMS (PLANS AND PROCEDURES):

MANAGERIAL ITEMS:

(1) Safety Audit and Review Committee review of Facility License Changes. (76-16)

PLANT STATUS:

No significant information

LAST IE SITE INSPECTION DATE: APRIL 14-17, 1980 *

INSPECTION REPORT NO: 50-285/80-08 *

SUMMARY

INSPECTION SUMMARY

* INSPECTION ON APRIL 14-17, 1980 (80-08): ROUTINE, UNANNOUNCED INSPECTION OF THE LICENSEE'S (1) IMPLEMENTATION OF NUREG 0578, CATEGORY A ITEMS; (2) FIRE PROTECTION COMMITMENTS FOR THE 1980 REFUELING OUTAGE; (3) PROGRESS ON IEB 79-02 AND IEB 79-14; AND (4) FOLLOW-UP ON ITEMS OF NONCOMPLIANCE AND DEVIATIONS. OF THE FOUR AREAS INSPECTED NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

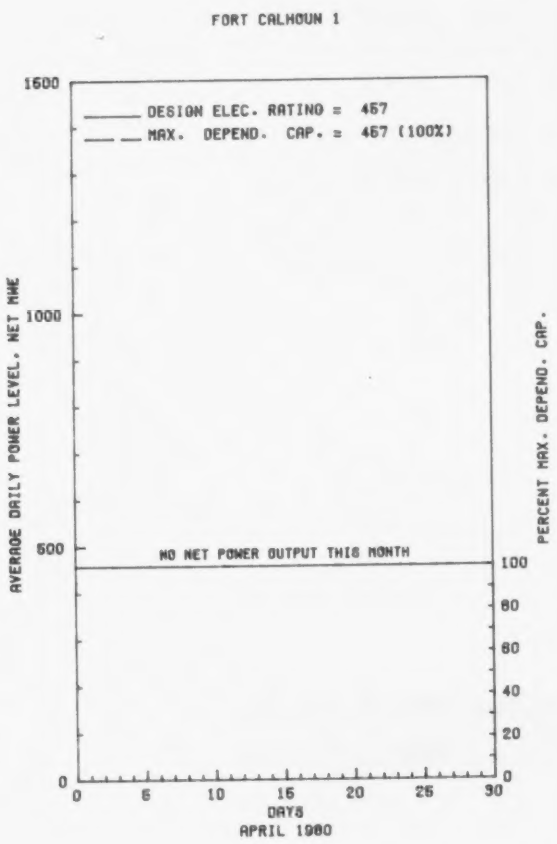
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE			

REVIEWED BY OMPA LOEB: ______ **DATE:** 6-12-80

REVIEWED BY NRR DOR: PC Wagner **DATE:** 6/13/80

FORT CALHOUN

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-285
2. Utility Contact: B. J. Hickle (402) 536-4413
3. Licensed Thermal Power (MWt): 1420
4. Nameplate Rating (Gross MWe): 591 x 0.85 = 502
5. Design Electrical Rating (Net MWe): 457
6. Maximum Dependable Capacity (Gross MWe): 481
7. Maximum Dependable Capacity (Net MWe): 457

ITEM 21 CALCULATED BY USING A WEIGHTED AVERAGE

8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	719.0	2,903.0	57,840.0
12. Hours Reactor Was Critical	.0	432.9	44,834.5
13. Reactor Reserve Shutdown Hours	.0	.0	1,309.5
14. Hours Generator On-Line	.0	430.5	43,881.6
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	574,731	53,809,758
17. Gross Electrical Energy (MWH)	0	190,604	17,850,834
18. Net Electrical Energy (MWH)	0	180,935	16,868,452
19. Unit Service Factor	.0	14.8	75.9
20. Unit Availability Factor	.0	14.8	75.9
21. Unit Capacity Factor (MDC Net)	.0	13.6	64.3
22. Unit Capacity Factor (DER Net)	.0	13.6	63.8
23. Unit Forced Outage Rate	.0	.0	4.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 05/15/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-01	01/18/80	S	719.0	C	4				REACTOR SHUTDOWN FOR ANNUAL REFUELING AND MAINTENANCE OUTAGE CONTINUES.

SUMMARY

THE UNIT REMAINED SHUTDOWN THROUGHOUT THE REPORT PERIOD FOR REFUELING AND MAINTENANCE.

LEGEND

[1] Type:
- F-Forced
- S-Scheduled

[2] Reason:
- A-Equipment Failure
- B-Maintenance or Test
- C-Refueling
- D-Regulatory Restriction
- E-Operator Training & License Examination
- F-Administrative
- G-Operational Error
- H-Other

[3] Method:
- 1-Manual
- 2-Manual Scram
- 3-Automatic Scram
- 4-Continuations
- 5-Load Reduction
- 9-Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

FORT ST. VRAIN

FACILITY DATA

Facility Description

1. Location (state/countv) (nearest population center)	Colorado/Weld 35 miles N of Denver (near Platteville)
2. Electric Reliability Council	Western Systems Coordinating Council
3. Type of Reactor	HTGR
4. Power Level (MWT)	842
5. Design Electrical Rating (MWE Net)	330
6. Date of Initial Criticality	January 31, 1974
7. Date of Commercial Operation	July 1, 1979
8. Condenser Cooling Method	Towers
9. Condenser Cooling Water Source	South Platte River/Shallow Wells

Utility & Contractor Information

10. Licensee	Public Service Company of Colorado
11. Corporate Address	P. O. Box 840 Denver, Colorado 80201
12. Corporate Contact	C. K. Millen Senior Vice President
13. Architect/Engineer	Sargent & Lundy
14. Nuclear Steam Supply System	General Atomic Company
15. Constructor	General Atomic Company

Regulatory Information

16. IE Region responsible	IV
17. IE Branch Chief	G. L. Madsen
18. IE Section Chief	T. F. Westerman
19. IE Resident Inspector	M. W. Dickerson (303) 785-2281
20. Licensing Project Manager	G. Kuzmycz
21. Docket Number	50-267
22. License Number/Date of Issue	DPR-34, 12/21/73
23. Public Document Room	Greeley Public Library City Complex Building Greeley, Colorado 80631

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO THE REQUIREMENTS OF TS AC 7.4 AND ADMINISTRATIVE PROCEDURE #14, PRIMARY COOLANT MOISTURE SCRAM CALIBRATIONS WERE PERFORMED UTILIZING A NONSPECIFIED MOISTURE DETECTOR AND RECORDS OF THE TRANSFER DEVICES USED FOR CALIBRATION WERE NOT MAINTAINED.
(8006 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

Areas related to potential equipment modification, which are under review and discussion with the NRC include: helium circulator auxiliaries; accident analysis relating to outlet temperature dispersion and orifice settings; hangers for 2" and smaller piping; and plant oscillations.

FACILITY ITEMS (PLANS AND PROCEDURES):

IMPLEMENTATION OF 10 CFR 73.55 PROVISIONS WERE IN EFFECT 3/21/80.

PLANT STATUS:

POWER LEVEL ≥ 45% WITH REACTOR OUTLET TEMPERATURE LESS THAN 1200 DEG F, WHILE ATTEMPTING TO LOCATE SOURCE OF OXIDANTS WHICH ARE RESTRICTING OPERATION TO AN OUTLET TEMPERATURE LESS THAN 1200 DEG F.

LAST IE SITE INSPECTION DATE: APRIL 2 - 3, 1980

INSPECTION REPORT NO: 50-267/80-07

SUMMARY

INSPECTION SUMMARY

INSPECTION MARCH 3, 1980 - MARCH 28, 1980 (80-06): ROUTINE, ANNOUNCED INSPECTION OF REVIEW OF EVENT REPORTS; SURVEILLANCE; MAINTENANCE; OPERATIONAL SAFETY VERIFICATION; REVIEW OF BULLETINS/CIRCULARS; AND REPORT REVIEWS. WITHIN THE SIX (6) AREAS INSPECTED, TWO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED (INFRACTION - FAILURE TO MEET TECHNICAL SPECIFICATION REQUIREMENTS FOR CORE REGIONAL TEMPERATURE RISE; INFRACTION - FAILURE TO FOLLOW PROCEDURES).

INSPECTION APRIL 2-3, 1980 (80-07): ROUTINE, UNANNOUNCED INSPECTION OF SECURITY PLAN AND DETECTION AIDS. WITHIN THE TWO AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

NRC POWER RESTRICTIONS: Restricted to 70% of Licensed Thermal Power.

REPORTS RECEIVED FROM LICENSEE

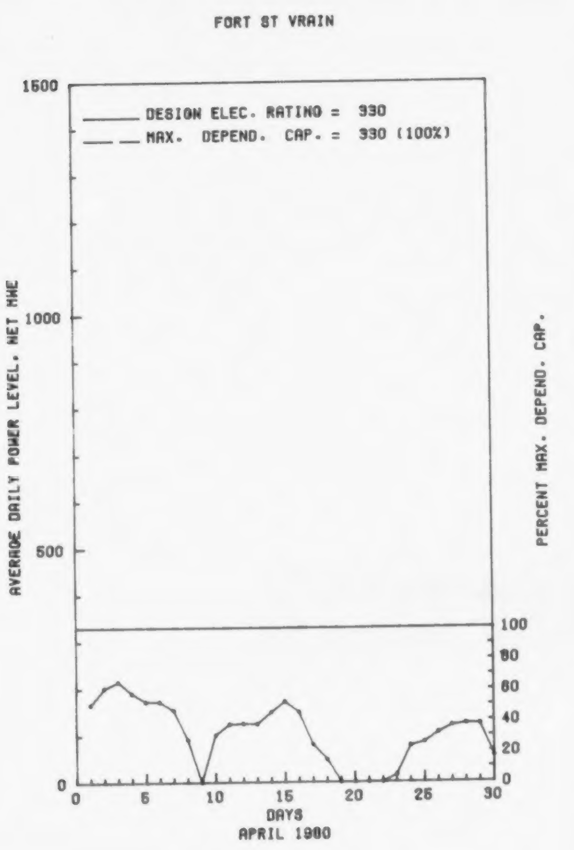
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-10 03L-0	3/13/80	4/2/80	"A" EMERGENCY DIESEL GENERATOR WOULD NOT LOAD PROPERLY. DEGRADED MODE OF LCO 4.6.1(D)
80-11 03L-0	3/6/80	4/2/80	DEWPOINT LIMITS OF LCO 4.2.11 EXCEEDED AND TOTAL OXIDANT LIMITS OF PRIMARY COOLANT IN EXCESS OF LCO 4.2.10 LIMITS
80-13 03L-0	3/10/80	4/9/80	SAMPLE FLOW FOR TWO PRIMARY COOLANT DEWPOINT MOISTURE MONITORS LESS THAN REQUIRED BY LCO 4.4.1
80-14 03L-0	3/11/80	4/9/80	SNUBBER BFS-228E FOUND TO BE INOPERABLE - LCO 4.3.10
80-15 03L-0	3/20/80	4/18/80	EMERGENCY FEEDWATER HEADER SUPPLY TO LOOP 2 HELIUM CIRCUALTOR WATER TURBINE DRIVE ISOLATED TO REPAIR LEAK - DEGRADED MODE LCO 4.2.2.(A)
80-16 03L-0	3/22/80	4/9/80	FOUR OF TWELVE HELIUM CIRCULATOR SEAL MALFUNCTION PRESSURE DIFFERENTIAL SWITCHES FOUND TO TRIP OUTSIDE LIMITS SPECIFIED BY LCO 4.4.1
80-17 03L-0	3/11/80	4/9/80	EFFLUENT SAMPLE REQUIRED BY SRNR 1.1 NOT TAKEN
80-18 03L-0	3/27/80	4/25/80	ONE OF THREE LOOP 2 HOT REHEAT HEADER ACTIVITY MONITORS INOPERABLE DUE TO FAILED OPERATIONAL AMPLIFIER - DEGRADED MODE LCO 4.4.1

REVIEWED BY OMPA LOEB: T.C. Cintula **DATE:** 6·12·80

REVIEWED BY NRR DOR: **DATE:** 6/13/1980

FORT ST. VRAIN

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-267
2. Utility Contact: J. W. GAHM (303) 785-2253
3. Licensed Thermal Power (MWt): 842
4. Nameplate Rating (Gross MWe): 403 x 0.85 = 343
5. Design Electrical Rating (Net MWe): 330
6. Maximum Dependable Capacity (Gross MWe): 342
7. Maximum Dependable Capacity (Net MWe): 330
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): 231
10. Reasons for Restrictions, If Any:
NRC RESTRICT 70% PEND RESOLUTION OF TEMP FLUCTUATIONS

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	719.0	2,903.0	7,320.0
12. Hours Reactor Was Critical	697.0	1,560.9	4,093.7
13. Reactor Reserve Shutdown Hours	.0	.0	0
14. Hours Generator On-Line	551.3	909.3	1,891.5
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	245,462	406,610	884,565
17. Gross Electrical Energy (MWH)	80,351	128,778	269,574
18. Net Electrical Energy (MWH)	74,589	117,636	241,220
19. Unit Service Factor	76.7	31.3	25.8
20. Unit Availability Factor	76.7	31.3	25.8
21. Unit Capacity Factor (MDC Net)	31.4	12.3	10.0
22. Unit Capacity Factor (DER Net)	31.4	12.3	10.0
23. Unit Forced Outage Rate	23.3	33.1	54.7

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-06	04/08/80	F	30.0	A	3				TURBINE GENERATOR TRIPPED DURING A LOOP 1 SHUTDOWN DUE TO INSTRUMENT PROBLEMS.
80-07	04/17/80	F	.8	F	5				REDUCED POWER DUE TO PRIMARY COOLANT MOISTURE AND ADMINISTRATIVE CONTROLS.
80-08	04/18/80	F	.7	A	3				SPURIOUS TRIP WHILE CHANGING INSTRUMENT MODULES.
80-09	04/18/80	F	118.7	A	3				TURBINE TRIPPED DURING A LOOP 1 SHUTDOWN DUE TO BUFFER-MID-BUFFER PROBLEM ON "A" PRIMARY COOLANT CIRCULATOR.
80-10	04/25/80	F	5.9	A	1				TURBINE TAKEN OFF LINE TO RECOVER FROM A LOOP SHUTDOWN.
80-11	04/30/80	F	12.4	A	1				TURBINE TAKEN OFF LINE TO RECOVER FROM A LOOP SHUTDOWN.

SUMMARY

THE UNIT OPERATED UNDER AN NRC RESTRICTION TO 231 NET MWE (70% POWER) PENDING RESOLUTION OF TEMPERATURE FLUCTUATIONS. ON 04/08/80 AND 04/18/80 THE TURBINE TRIPPED DUE TO INSTRUMENT PROBLEMS DURING A LOOP 1 SHUTDOWN: A SPURIOUS TRIP ALSO OCCURRED ON 04/18/80 WHILE CHANGING INSTRUMENT MODULES. THE TURBINE WAS TAKEN OFF LINE ON 04/25/80 AND 04/30/80 TO RECOVER FROM THE LOOP SHUTDOWN. A BRIEF LOAD REDUCTION OCCURRED ON 04/17/80 DUE TO PRIMARY COOLANT MOISTURE AND ADMINISTRATIVE CONTROLS.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

GINNA

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	New York/Wayne 15 miles NE of Rochester
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	1520
5.	Design Electrical Rating (MWE Net)	470
6.	Date of Initial Criticality	November 8, 1969
7.	Date of Commercial Operation	July 1970
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Lake Ontario

Utility & Contractor Information

10.	Licensee	Rochester Gas & Electric Corporation
11.	Corporate Address	89 East Avenue Rochester, New York
12.	Corporate Contact	Leon D. White, Jr., Vice President Electric and Steam Production
13.	Architect/Engineer	Gilbert
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Section Chief	H. B. KISTER
19.	IE Resident Inspector	D. JOHNSON
20.	Licensing Project Manager	S. J. Nowicki
21.	Docket Number	50-244
22.	License Number/Date of Issue	DPR-18, 9/19/69
23.	Public Document Room	Rochester Public Library Business and Social Science Division 115 South Avenue Rochester, New York 14604

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO TS 6.8.1.A, PROCEDURE CP-2.5.1, AND A-12-2, DURING ALIGNMENT ON CONTROL ROD BANK A ON DECEMBER 20, 1979, DATA WAS NOT RECORDED FOR INDIVIDUAL RODS AS REQUIRED BY PROCEDURE; AND ON AUGUST 7, 1979 DATA FOR CONTROL RODS WAS RECORDED FROM COMPUTER RATHER THAN BENCH BOARD AS REQUIRED BY PROCEDURE. DATA FOR CONTAINMENT SPRAY PUMP LOW FLOW ALARM WAS RECORDED OUT OF SPECIFICATION ON NOVEMBER 8, 1979. SUPERVISORY REVIEW WAS INADEQUATE IN THAT ABOVE ERRORS WERE OBSERVED DURING SUPERVISORY REVIEW.
(8002 DEFICIENCY)

CONTRARY TO THE LICENSEE'S LETTER TO NRR DATED DECEMBER 28, 1979, THAT STATED "TRAINING OF OPERATORS ON THESE REVISED EMERGENCY PROCEDURES HAS BEEN COMPLETED." 18 OUT OF 37 OPERATORS WERE NOT TRAINED.
(8004 DEVIATION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS:

- Fuel shuffling completed on 4/23/80.
  Startup scheduled for 5/14/80.
- LAST IE SITE INSPECTION DATE: 4/25/80
- INSPECTION REPORT NO: 50-244/80-05

SUMMARY

INSPECTION SUMMARY

- 50-244/80-02 - Feb 19-22: Routine, unannounced inspection (29 hours onsite) by a region-based inspector of action on previous inspection findings; administrative control of safety-related equipment required by Technical Specifications; calibrations required by Technical Specifications of components and equipment associated with safety-related systems and/or functions; calibration and control of test equipment; witnessing of calibrations; technician qualifications; and control room tour. Noncompliances: One (failure to record required data; acceptance of out of tolerance setpoint; and inadequate supervisory data review).
- 50-244/80-04 - Mar 3-7: Routine, announced inspection to discuss establishment of an on-site office for a full time NRC inspector at the facility, to turn over the assignment of the principal IE inspector and to review the revised small break loss of coolant emergency procedures including operator training related to these procedures. Noncompliances: None. Deviations: One (failure to meet commitment).

REPORTS RECEIVED FROM LICENSEE

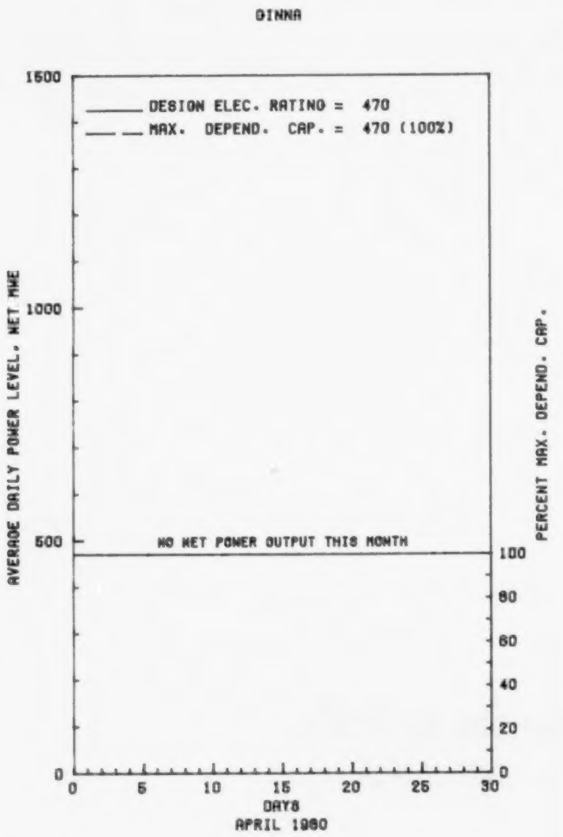
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-002/ 03L	03/31/80	04/14/80	Boron sample frequency for safety injection accumulators exceeded Technical Specification requirements

REVIEWED BY OMPA LOEB: T.C. Cintula **DATE:** 6·12·80

REVIEWED BY NRR DOR: S. J. Nowicki **DATE:** 6·13·80

GINNA

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-244
2. Utility Contact: A.E. MCNAMARA (315) 524-4446
3. Licensed Thermal Power (MWt): 1520
4. Nameplate Rating (Gross MWe): 608 x 0.85 = 517
5. Design Electrical Rating (Net MWe): 470
6. Maximum Dependable Capacity (Gross MWe): 490
7. Maximum Dependable Capacity (Net MWe): 470
8. If Changes Occur in Capacity Ratings Above Since Last Report. Give Reasons: NONE
9. Power Level To Which Restricted. If Any (Net MWe): NONE
10. Reasons for Restrictions. If Any: NONE

CAPACITY FACTORS CALCULATED WITH A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	719.0	2,903.0	91,415.0
12. Hours Reactor Was Critical	.0	2,114.5	69,794.3
13. Reactor Reserve Shutdown Hours	.0	.0	1,621.8
14. Hours Generator On-Line	.0	2,114.5	68,071.3
15. Unit Reserve Shutdown Hours	.0	.0	8.5
16. Gross Thermal Energy (MWH)	0	3,118,656	91,394,458
17. Gross Electrical Energy (MWH)	0	1,021,206	29,786,997
18. Net Electrical Energy (MWH)	0	970,811	28,132,359
19. Unit Service Factor	.0	72.8	74.5
20. Unit Availability Factor	.0	72.8	74.5
21. Unit Capacity Factor (MDC Net)	.0	71.2	67.9
22. Unit Capacity Factor (DER Net)	.0	71.2	67.9
23. Unit Forced Outage Rate	.0	.0	9.6

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period. Estimated Date of Startup: 05/17/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
1	03/29/80	S	719.0	C	4				ANNUAL REFUELING AND MAINTENANCE SHUTDOWN CONTINUED.

SUMMARY

THE UNIT REMAINED SHUTDOWN THROUGHOUT THE REPORT PERIOD FOR REFUELING.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

HADDAM NECK

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1. Location (state/county) (nearest population center)	Connecticut/Middlesex 13 miles E of Meriden
2. Electric Reliability Council	Northeast Power Coordinating Council
3. Type of Reactor	PWR
4. Power Level (MWT)	1825
5. Design Electrical Rating (MWE Net)	575
6. Date of Initial Criticality	July 24, 1967
7. Date of Commercial Operation	January 1, 1968
8. Condenser Cooling Method	Once Through
9. Condenser Cooling Water Source	Connecticut River

Utility & Contractor Information

10. Licensee	Connecticut Yankee Atomic Power Co.
11. Corporate Address	P. O. Box 270 Hartford, Connecticut
12. Corporate Contact	D. Switzer, President
13. Architect/Engineer	Stone & Webster
14. Nuclear Steam Supply System	Westinghouse
15. Constructor	Stone & Webster

Regulatory Information

16. IE Region responsible	I
17. IE Section Chief	E. J. Brunner
18. IE Resident Inspector	R. Keimig
19. IE Principal Inspector	T. H. Smith
20. Licensing Project Manager	R. Caruso
21. Docket Number	50 213
22. License Number/Date of Issue	DPR-61, 12/27/74
23. Public Document Room	Russell Library 119 Broad Street Middletown, Connecticut 06457

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS:

+ Plant shutdown on May 3, 1980 for an eight week refueling outage.

+ LAST IE SITE INSPECTION DATE: 5/4-5/80

+ INSPECTION REPORT NO: 50-213/80-09

SUMMARY

INSPECTION SUMMARY

+ None.

REPORTS RECEIVED FROM LICENSEE

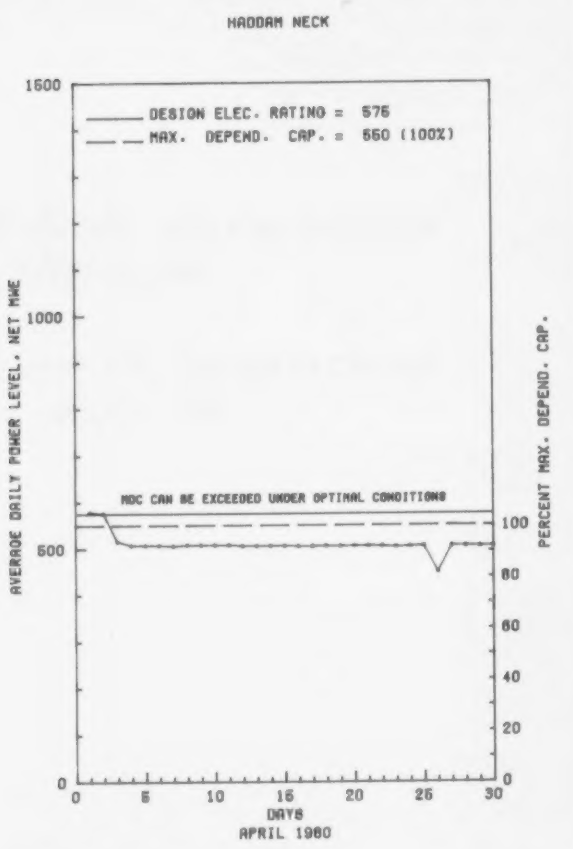
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-006/ 03L	03/06/80	03/20/80	Broken linkage coupling on car and bypass damper
80-007/ 03L	02/27/80	03/25/80	Drain line leak caused surrounding sand to become contaminated
80-008/ 03L	02/26/80	03/25/80	Stack radiation monitor system failed due to bent module connector

REVIEWED BY OMPA LOEB: T. C. Cintula **DATE:** 6·12·80

REVIEWED BY NRR DOR: R Caruso **DATE:** 6/16/80

HADDAM NECK

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-213
2. Utility Contact: C. Kokoszka (203) 267-2556
3. Licensed Thermal Power (MWt): 1825
4. Nameplate Rating (Gross MWe): 667 x 0.9 = 600
5. Design Electrical Rating (Net MWe): 575
6. Maximum Dependable Capacity (Gross MWe): 577
7. Maximum Dependable Capacity (Net MWe): 550
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): 484
10. Reasons for Restrictions, If Any:
CHECKING TURBINE DISK CRACKING POSSIBILITIES.

CAPACITY FACTORS CALCULATED WITH A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	719.0	2,903.0	108,195.0
12. Hours Reactor Was Critical	719.0	2,891.6	94,088.3
13. Reactor Reserve Shutdown Hours	.0	11.4	1,164.0
14. Hours Generator On-Line	719.0	2,889.3	89,829.5
15. Unit Reserve Shutdown Hours	.0	.0	369.9
16. Gross Thermal Energy (MWH)	1,145,726	5,004,699	155,145,848
17. Gross Electrical Energy (MWH)	384,639	1,663,586	51,018,052
18. Net Electrical Energy (MWH)	365,821	1,586,212	48,540,975
19. Unit Service Factor	100.0	99.5	83.1
20. Unit Availability Factor	100.0	99.5	83.4
*21. Unit Capacity Factor (MDC Net)	92.5	99.3	83.1
22. Unit Capacity Factor (DER Net)	88.5	95.0	77.3
23. Unit Forced Outage Rate	.0	.5	7.1

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
REFUELING, MAY 3, 1980, APPROX 8 TO 12 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence

POWER LIMITED TO 88% OF FULL POWER FROM 4/3/80 TO 5/3/80 DUE TO SUSPECTED CRACKS IN LP TURBINE DISC. TURBINE TO BE INSPECTED DURING REFUELING SHUTDOWN WHICH WILL START ON 5/3/80.

SUMMARY

OPERATION WAS AT NEAR FULL POWER THROUGHOUT THE REPORT PERIOD.

LEGEND

[1] Type:
F-Forced
S-Scheduled

[2] Reason:
A-Equipment Failure
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error
H-Other

[3] Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Continuations
5-Load Reduction
9-Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

HATCH 1

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Georgia/Appling 11 miles N of Baxley
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2436
5.	Design Electrical Rating (MWE Net)	777
6.	Date of Initial Criticality	September 12, 1974
7.	Date of Commercial Operation	December 31, 1975
8.	Condenser Cooling Method	Once Through With Forced Draft Cooling Towers
9.	Condenser Cooling Water Source	Altamaha River

Utility & Contractor Information

10.	Licensee	Georgia Power Company
11.	Corporate Address	270 Peachtree Street, NE Atlanta, Georgia 30303
12.	Corporate Contact	J. H. Miller Executive Vice President
13.	Architect/Engineer	Southern Services, Inc.
14.	Nuclear Steam Supply System	GE
15.	Constructor	Georgia Power Company

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. Lewis
18.	IE Section Chief	H. C. Dance
19.	IE Resident Inspector	R. F. Rogers (912) 367-9881
20.	Licensing Project Manager	D. Verrelli
21.	Docket Number	50-321
22.	License Number/Date of Issue	DPR-57, 8/6/74
23.	Public Document Room	Appling County Public Library Parker Street Baxley, Georgia 31513

INSPECTION STATUS

ENFORCEMENT STATUS

THE HATCH QUALITY ASSURANCE MANUAL INTRODUCTION IDENTIFIES THAT THE MANUAL WAS WRITTEN SO AS TO ENSURE THE QUALITY ASSURANCE PROGRAM FULLY COMPLIES WITH ANSI N18.7-1976, "ADMINISTRATIVE CONTROLS AND QUALITY ASSURANCE FOR THE OPERATIONAL PHASE OF NUCLEAR POWER PLANTS." ANSI N18.7-1976, SECTION 5.2.2STATES IN PART: "PROCEDURAL STEPS FOR WHICH ACTIONS SHOULD BE COMMITTED TO MEMORY INCLUDE, FOR EXAMPLE, IMMEDIATE ACTIONS IN EMERGENCY PROCEDURES." NUREG-0094, "A GUIDE FOR THE LICENSING OF FACILITY OPERATORS, INCLUDING SENIOR OPERATORS," SECTION IV.F FURTHER STATES THAT THE APPLICANT MUST DEMONSTRATE COMPLETE UNDERSTANDING OF THE SYMPTOMS, AUTOMATIC ACTIONS AND IMMEDIATE ACTION STEPS SPECIFIED BY OFF-NORMAL OR EMERGENCY OPERATING PROCEDURES. ON FEBRUARY 20, 1980, THE INSPECTOR DISCOVERED THAT THE LICENSED PERSONNEL AT PLANT HATCH WERE UNABLE TO CONVEY ALL THE IMMEDIATE OPERATOR ACTIONS SPECIFIED IN THE FACILITY'S EMERGENCY SMALL BREAK LOSS OF COOLANT ACCIDENT PROCEDURES HNP-1902 AND HNP-1904.
(8006 DEVIATION)

AS REQUIRED, IN PART, BY SECTIONS 3.1.2.1.2 AND 3.1.2.1.3 OF APPENDIX B TECHNICAL SPECIFICATIONS, MONITORING PROGRAMS FOR ENTRAINMENT OF ICHTHYOPLANKTON AND IMPINGEMENT OF ORGANISMS, RESPECTIVELY, WERE TO COMMENCE ON JANUARY 2, 1979. THE SUBJECT MONITORING PROGRAMS WERE COMMENCED FEBRUARY 1, 1979, WITHOUT PRIOR NOTIFICATION OF NOR APPROVAL BY THE NRC.
(8012 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

◆ PLANT IS OPERATING AT FULL POWER.

◆ LAST IE SITE INSPECTION DATE: MARCH 31 - APRIL 2, 1980

◆ INSPECTION REPORT NO: 50-321/80-18

SUMMARY

INSPECTION SUMMARY

◆ INSPECTION JANUARY 7-11 (80-03): INCLUDED REVIEW OF SECURITY PLAN; SECURITY PROGRAM AUDIT; SECURITY ORGANIZATION; PHYSICAL BARRIERS (PROTECTED AREA); PHYSICAL BARRIERS (VITAL AREA); LIGHTING; ACCESS CONTROL (VITAL AREA); AND IE BULLETIN 79-16. THE INSPECTION INVOLVED 24 INSPECTOR-HOURS ON SITE. THE INSPECTION WAS BEGUN DURING AN OFFSHIFT PERIOD; 6 INSPECTION HOURS WERE ACCOMPLISHED DURING OFFSHIFT PERIODS. DURING THE INSPECTION, AN INSPECTOR ASSISTED AN NRC INVESTIGATOR WHO WAS ON SITE. THE LICENSEE WAS FOUND TO BE IN COMPLIANCE WITH NRC REQUIREMENTS WITHIN THE 8 AREAS EXAMINED DURING THE INSPECTION EXCEPT FOR THE FOLLOWING THREE ITEMS: (1) SECURITY ORGANIZATION - FAILURE TO FOLLOW PROCEDURES FOR DOCUMENTING SECURITY INCIDENTS (INFRACTION); (2) PHYSICAL BARRIERS (VITAL AREA) - A VITAL AREA BARRIER DID NOT MEET THE SPECIFICATIONS OF THE LICENSEE'S SECURITY PLAN (INFRACTION); (3) ACCESS CONTROL (VITAL AREA) - FAILURE TO PERFORM VITAL AREA PATROLS AS COMMITTED (INFRACTION).

INSPECTION MARCH 11-14 (80-07): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 13 INSPECTOR-HOURS ON SITE IN THE AREAS OF IE BULLETIN 79-14, "SEISMIC ANALYSIS FOR AS-BUILT SAFETY-RELATED PIPING SYSTEMS" AND LICENSEE ACTION ON PREVIOUS INSPECTION FINDINGS. OF THE TWO AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION MARCH 24-26 (80-09): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 17 INSPECTOR-HOURS ON SITE IN THE AREAS OF IE BULLETIN 79-14, "SEISMIC ANALYSIS FOR AS-BUILT SAFETY-RELATED PIPING SYSTEMS." AND LICENSEE ACTIONS ON PREVIOUS INSPECTION FINDINGS. OF THE TWO AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION FEBRUARY 2 - MARCH 7 (80-11): THIS INSPECTION INVOLVED 56 INSPECTOR-HOURS ON SITE IN THE AREAS OF TECHNICAL SPECIFICATION COMPLIANCE. REPORTABLE OCCURRENCES, HOUSEKEEPING, OPERATOR PERFORMANCE, OVERALL PLANT OPERATIONS, QUALITY ASSURANCE PRACTICES, STATION AND CORPORATE MANAGEMENT PRACTICES, CORRECTIVE AND PREVENTATIVE MAINTENANCE ACTIVITIES, SITE SECURITY PROCEDURES, RADIATION CONTROL ACTIVITIES AND SURVEILLANCE ACTIVITIES. OF THE ELEVEN AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN SIX AREAS; SIX ITEMS OF NONCOMPLIANCE WERE FOUND IN FIVE AREAS (INFRACTION - FAILURE TO FOLLOW PROCEDURE; INFRACTION - FAILURE TO OPERATE THE RSCS PROPERLY; INFRACTION - FAILURE TO MAKE IMMEDIATE NOTIFICATION TO THE NRC; INFRACTION - FAILURE TO LIMIT ROD MOTION WITH RSCS INOPERABLE; INFRACTION - FAILURE TO EVALUATE PROCEDURE CHANGE; INFRACTION - FAILURE TO PROVIDE UPDATED PLANT DRAWINGS IN THE CONTROL ROOM).

INSPECTION MARCH 10-19 (80-12): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 33 INSPECTOR-HOURS ON SITE IN THE AREAS OF RADIOLOGICAL AND NONRADIOLOGICAL ENVIRONMENTAL PROTECTION INCLUDING MANAGEMENT CONTROLS, REVIEW OF PREVIOUS ENFORCEMENT MATTERS, REVIEW OF ENVIRONMENTAL SURVEILLANCE, REVIEW OF SPECIAL SURVEILLANCE AND STUDY ACTIVITIES, STATUS REVIEW OF LICENSEE EVENT REPORTS, SELECTED SOIL AND GROUNDWATER INDEPENDENT SAMPLING. OF THE SIX AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN FIVE AREAS; ONE ITEM OF NONCOMPLIANCE WAS FOUND IN ONE AREA (DEFICIENCY - ENVIRONMENTAL MONITORING).

INSPECTION MARCH 11-14 AND 25-28 (80-13): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 26 INSPECTOR-HOURS ON SITE IN THE AREAS OF INSERVICE INSPECTION ACTIVITIES (UNIT 2) AND MAINTENANCE WELDING PROGRAM (UNIT 1 AND 2). OF THE 2 AREAS INSPECTED, 2 ITEMS OF NONCOMPLIANCE WERE FOUND IN 2 AREAS (DEFICIENCY - FAILURE TO APPROVE ISI PLAN; AND INFRACTION - FAILURE TO CERTIFY WELDING AND NDE INSPECTORS).

INSPECTION APRIL 2-4 (80-15): THIS SPECIAL, ANNOUNCED INSPECTION INVOLVED 9 INSPECTOR-HOURS ON SITE IN THE AREA OF INSTALLED EQUIPMENT REVIEW WITH RESPECT TO THE IE BULLETIN 79-01B RESPONSE. OF THE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION MARCH 26-28 (80-16): INCLUDED REVIEW OF LICENSEE COMMITMENTS MADE DURING THE JANUARY 25, 1980 ENFORCEMENT MEETING, PHYSICAL BARRIERS (PROTECTED AREA), PHYSICAL BARRIERS (VITAL AREA), LIGHTING, ACCESS CONTROL (IDENTIFICATION, AUTHORIZATION AND BADGING), ACCESS CONTROL (SEARCH), AND COMMUNICATIONS. THE INSPECTION INVOLVED 15 INSPECTOR-HOURS ON SITE BY 2 NRC INSPECTORS. THE INSPECTION WAS BEGUN DURING AN OFFSHIFT PERIOD; 4 INSPECTION HOURS WERE ACCOMPLISHED DURING OFFSHIFT PERIODS. THE LICENSEE WAS FOUND TO BE IN COMPLIANCE WITH NRC REQUIREMENTS WITHIN THE 7 AREAS EXAMINED DURING THE INSPECTION EXCEPT FOR THE FOLLOWING 2 ITEMS: (1) ACCESS CONTROL - FAILURE TO CONTROL ALL POINTS OF PERSONNEL ACCESS INTO VITAL AREAS AND TO VITAL EQUIPMENT (INFRACTION); (2) LIGHTING - FAILURE TO MAINTAIN LIGHTING EQUIPMENT AS COMMITTED TO IN THE SECURITY PLAN (DEFICIENCY); (3) CERTIFICATION OF BULLET RESISTANT STRUCTURES (UNRESOLVED).

INSPECTION MARCH 31 - APRIL 2 (80-18): THIS SPECIAL, ANNOUNCED INSPECTION INVOLVED 16 INSPECTOR-HOURS ON SITE IN THE AREAS OF FOLLOWUP OF RESIDENT-INSPECTOR IDENTIFIED ITEMS OF NONCOMPLIANCE, UNIT STARTUP OPERATIONS AND POST-MODIFICATION TESTING. OF THE THREE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN TWO AREAS; ONE APPARENT ITEM OF NONCOMPLIANCE WAS FOUND IN ONE AREA (INFRACTION - FAILURE TO PERFORM POST-MODIFICATION TESTING).

ENFORCEMENT MEETING APRIL 16 (80-20): DISCUSSED LICENSEE'S ENFORCEMENT HISTORY, OPERATIONAL EVENTS, AND SECURITY ISSUES. GEORGIA POWER COMPANY REPRESENTATIVES ACKNOWLEDGED THE NRC CONCERNS AND INDICATED THAT CORRECTIVE ACTION WOULD BE IMPLEMENTED IN THE AREAS IDENTIFIED.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FPOM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
80-021/ 03L-0	02/21/80	03/17/80	30 DAY LER	CONTAINMENT INTEGRITY VIOLATED REACTOR BLDG TO CONTROL BLDG AIRLOCK.
80-022/ 03L-0	02/22/80	03/11/80	30 DAY LER	LACK OF ELECTRONIC ANNUNCIATION REQUIRE ISSUE PART 21 REPORT.
80-024/ 03L-0	02/29/80	03/17/80	30 DAY LER	1C DIESEL GENERATOR MANUAL VOLTAGE REGULATOR INOPERATIVE.
80-025/ 03L-0	03/01/80	03/25/80	30 DAY LER	REACTOR CORE ISOLATION COOLING SYSTEM FLOW SWITCH LESS THAN TECHNICAL SPECIFICATION.
80-026/ 03L-0	02/18/80	03/17/80	30 DAY LER	PLANT SERVICE WATER PUMP HEAD FAILED INSERVICE TEST.

REVIEWED BY OMPA LOEB: Earl Hill

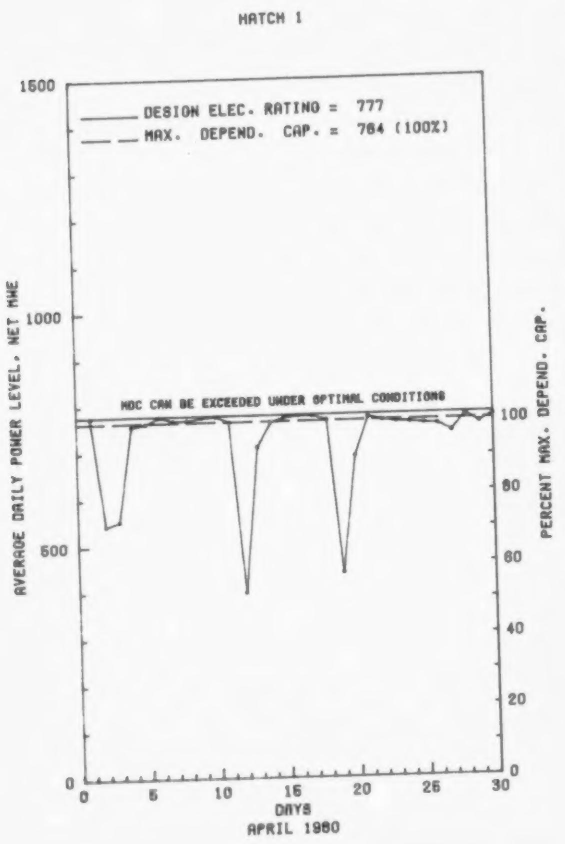
DATE: 6/12/80

REVIEWED BY NRR DOR: Wm Dennis

DATE: 6/13/80

HATCH 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-321
2. Utility Contact: J. Roden (912) 367-7781
3. Licensed Thermal Power (MWt): 2436
4. Nameplate Rating (Gross MWe): 1000 x 0.85 = 850
5. Design Electrical Rating (Net MWe): 777
6. Maximum Dependable Capacity (Gross MWe): 797
7. Maximum Dependable Capacity (Net MWe): 764
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
   NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
   NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	719.0	2,903.0	37,967.0
12. Hours Reactor Was Critical	719.0	2,683.8	28,481.8
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	719.0	2,608.8	26,877.2
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,634,360	5,940,570	56,293,605
17. Gross Electrical Energy (MWH)	524,810	1,957,860	18,228,800
18. Net Electrical Energy (MWH)	520,541	1,891,383	17,353,316
19. Unit Service Factor	100.0	89.9	70.8
20. Unit Availability Factor	100.0	89.9	70.8
21. Unit Capacity Factor (MDC Net)	94.8	85.3	59.8
22. Unit Capacity Factor (DER Net)	93.2	83.9	58.8
23. Unit Forced Outage Rate	.0	10.1	15.6

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
   NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-10	04/02/80	F	.0	D	5	80-32	SE	ZZZZZZ	REDUCTION DUE TO HIGH DRYWELL 02 CONTENT.
80-11	04/11/80	F	.0	A	5		WA	PIPEXX	REDUCTION TO REPAIR SERVICE WATER LEAK IN BUSS DUCT COOLING SYSTEM.
80-12	04/18/80	S	.0	H	5		ZZ	CONROD	REDUCTION TO PERFORM TURBINE TEST AND CONTROL ROD SEQUENCE EXCHANGE.

SUMMARY

OPERATION WAS AT NEAR FULL POWER DURING THE REPORT PERIOD, WITH LOAD REDUCTIONS DUE TO HIGH DRYWELL OXYGEN CONTENT, FOR REPAIR OF A SERVICE WATER LEAK, AND FOR CONTROL ROD ADJUSTMENT.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

HATCH 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1. Location (state/county) (nearest population center)	Georgia/Appling 11 miles N of Baxley
2. Electric Reliability Council	Southeastern Electric Reliability Council
3. Type of Reactor	BWR
4. Power Level (MWT)	2436
5. Design Electrical Rating (MWE Net)	784
6. Date of Initial Criticality	July 4, 1978
7. Date of Commercial Operation	September 5, 1979
8. Condenser Cooling Method	Cooling Towers
9. Condenser Cooling Water Source	Altamaha River

Utility & Contractor Information

10. Licensee	Georgia Power Company
11. Corporate Address	270 Peach Street, NE Atlanta, Georgia 30303
12. Corporate Contact	J. H. Miller Executive Vice President
13. Architect/Engineer	Southern Services, Inc./Bechtel
14. Nuclear Steam Supply System	GE
15. Constructor	Georgia Power Company

Regulatory Information

16. IE Region responsible	II
17. IE Branch Chief	R. C. Lewis
18. IE Section Chief	H. C. Dance
19. IE Resident Inspector	R. F. Rogers (912) 367-9881
20. Licensing Project Manager	D. Verrelli
21. Docket Number	50-366
22. License Number/Date of Issue	NPF-5/June 13, 1978
23. Public Document Room	Appling County Public Library Parker Street Baxley, Georgia 31513

INSPECTION STATUS

ENFORCEMENT STATUS

THE HATCH QUALITY ASSURANCE MANUAL INTRODUCTION IDENTIFIES THAT THE MANUAL WAS WRITTEN SO AS TO ENSURE THE QUALITY ASSURANCE PROGRAM FULLY COMPLIES WITH ANSI N18.7-1976, "ADMINISTRATIVE CONTROLS AND QUALITY ASSURANCE FOR THE OPERATIONAL PHASE OF NUCLEAR POWER PLANTS." ANSI N18.7-1976, SECTION 5.2.2STATES IN PART: "PROCEDURAL STEPS FOR WHICH ACTIONS SHOULD BE COMMITTED TO MEMORY INCLUDE, FOR EXAMPLE, IMMEDIATE ACTIONS IN EMERGENCY PROCEDURES." NUREG-0094, "A GUIDE FOR THE LICENSING OF FACILITY OPERATORS, INCLUDING SENIOR OPERATORS," SECTION IV.F FURTHER STATES THAT THE APPLICANT MUST DEMONSTRATE COMPLETE UNDERSTANDING OF THE SYMPTOMS, AUTOMATIC ACTIONS AND IMMEDIATE ACTION STEPS SPECIFIED BY OFF-NORMAL OR EMERGENCY OPERATING PROCEDURES. ON FEBRUARY 28, 1980, THE INSPECTOR DISCOVERED THAT THE LICENSED PERSONNEL AT PLANT HATCH WERE UNABLE TO CONVEY ALL THE IMMEDIATE OPERATOR ACTIONS SPECIFIED IN THE FACILITY'S EMERGENCY SMALL BREAK LOSS OF COOLANT ACCIDENT PROCEDURES HNP-1902 AND HNP-1904.
(8006 DEVIATION)

AS REQUIRED, IN PART, BY SECTIONS 3.1.2.1.2 AND 3.1.2.1.3 OF APPENDIX B TECHNICAL SPECIFICATIONS, MONITORING PROGRAMS FOR ENTRAINMENT OF ICHTHYOPLANKTON AND IMPINGEMENT OF ORGANISMS, RESPECTIVELY, WERE TO COMMENCE ON JANUARY 2, 1979. THE SUBJECT MONITORING PROGRAMS WERE COMMENCED FEBRUARY 1, 1979, WITHOUT PRIOR NOTIFICATION OF NOR APPROVAL BY THE NRC.
(8012 DEFICIENCY)

AS REQUIRED BY CRITERION V OF APPENDIX B TO 10 CFR 50 AND AS IMPLEMENTED BY PARAGRAPH 17.2.5 OF THE FSAR, "ACTIVITIES AFFECTING QUALITY SHALL BE PRESCRIBED BY DOCUMENTED INSTRUCTIONS, PROCEDURES, DRAWINGS....AND SHALL BE ACCOMPLISHED IN ACCORDANCE WITH THESE INSTRUCTIONS, PROCEDURES, AND DRAWINGS". PARAGRAPH B.1.D(2) OF PLANT PROCEDURE HNP-824, THE APPLICABLE PROCEDURE FOR CONTROL OF SPECIAL PROCESSES, REQUIRES THAT, "PERSONNEL PERFORMING NONDESTRUCTIVE TESTING SHALL BE CERTIFIED IN ACCORDANCE WITH HNP-823 AND SNT-TC-1A". ON MARCH 27, 1980, WELDING RECORDS REVEALED THAT INSPECTION PERSONNEL WHO HAD PERFORMED WELDING AND NDE INSPECTIONS ON AMODIFICATION TO THE CONTROL ROD DRIVE SYSTEM WERE CERTIFIED BY DANIEL CONSTRUCTION COMPANY AND/OR ALABAMA POWER COMPANY IN LIEU OF CERTIFICATION IN ACCORDANCE WITH PLANT PROCEDURE HNP-823 AND THE REQUIREMENTS OF PARAGRAPHS 9 AND 10 OF SNT-TC-1A.
(8013 INFRACTION)

AS REQUIRED BY CRITERION V OF APPENDIX B TO 10 CFR 50 AND AS IMPLEMENTED BY PARAGRAPH 17.2.5 OF THE FSAR, "ACTIVITIES AFFECTING QUALITY SHALL BE PRESCRIBED BY DOCUMENTED INSTRUCTIONS, PROCEDURES, DRAWINGS....AND SHALL BE ACCOMPLISHED IN ACCORDANCE WITH THESE INSTRUCTIONS, PROCEDURES, AND DRAWINGS". PARAGRAPH F OF PLANT PROCEDURE HNP-904, THE APPLICABLE PROCEDURE FOR ADMINISTRATION OF THE INSERVICE INSPECTION PROGRAM, REQUIRES THAT THE WRITTEN DOCUMENT DETAILING THE PROPOSED WORK TO BE DONE BE REVIEWED AND APPROVED PRIOR TO THE OUTAGE. DURING MARCH 18-21, 1980, INSPECTIONS WERE BEING PERFORMED AND THE PROJECT PLAN DETAILING THE WORK TO BE DONE HAD NOT BEEN APPROVED BY THE LICENSEE.
(8013 VIOLATION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

- PLANT IS OPERATING AT FULL POWER.
- LAST IE SITE INSPECTION DATE: MARCH 31 - APRIL 2, 1980
- INSPECTION REPORT NO: 50-366/80-18

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
80-005/ 03L-0	01/29/80	02/13/80	30 DAY LER	ROD SEQUENCE CONTROL SYSTEM INOPERABLE.
80-017/ 03L-0	02/19/80	03/13/80	30 DAY LER	B DRYWELL HYDROGEN-OXYGEN ANALYZER VACUUM PUMP FAILED.
80-018/ 03L-0	02/20/80	03/11/80	30 DAY LER	TORUS-DRYWELL VACUUM BREAK VALVE FAILED SURVEILLANCE.
80-019/ 03L-0	02/27/80	03/24/80	30 DAY LER	A DRYWELL HYDROGEN-OXYGEN ANALYZER VACUUM PUMP NOT WORKING.
80-020/ 03L-0	02/29/80	03/24/80	30 DAY LER	HIGH PRESSURE COOLANT INJECTION EXHAUST DIAPHRAGM PRESSURE SWITCH FAIL SURVEILLANCE.
80-022/ 01T-0	03/05/80	03/17/80	14 DAY LER	MSIVS FAILED PERIOD TYPE B AND C LEAK TEST.
80-023/ 03L-0	01/29/80	03/11/80	30 DAY LER	ROD 46-43 NOT PULLED IN REQUIRED SEQUENCE.
80-025/ 03L-0	03/08/80	03/25/80	30 DAY LER	RESIDUAL HEAT REMOVAL PUMP START TIMER CALIBRATION OUT OF SPECIFICATION.
80-027/ 03L-0	03/15/80	03/25/80	30 DAY LER	PLANT SERVICE WATER PUMP ISOLATION VALVE FAILED SURVEILLANCE.
80-028/ 03L-0	02/28/80	03/25/80	30 DAY LER	B DRYWELL HYDROGEN-OXYGEN ANALYZER SYSTEM IN LEAKAGE.
80-029/ 01T-0	03/25/80	04/03/80	14 DAY LER	BATTERY CHARGER FAILED VOLTAGE REQUIREMENTS.
80-031/ 01T-0	03/27/80	04/07/80	14 DAY LER	BATTERY CHARGERS FAILED LOAD REQUIREMENTS.

SUMMARY

INSPECTION SUMMARY

- INSPECTION JANUARY 7-11 (80-03): INCLUDED REVIEW OF SECURITY PLAN; SECURITY PROGRAM AUDIT; SECURITY ORGANIZATION; PHYSICAL BARRIERS (PROTECTED AREA); PHYSICAL BARRIERS (VITAL AREA); LIGHTING; ACCESS CONTROL (VITAL AREA); AND IE BULLETIN 79-16. THE INSPECTION INVOLVED 23 INSPECTOR-HOURS ON SITE. THE INSPECTION WAS BEGUN DURING AN OFFSHIFT PERIOD; 6 INSPECTION HOURS WERE ACCOMPLISHED DURING OFFSHIFT PERIODS. DURING THE INSPECTION, AN INSPECTOR ASSISTED AN NRC INVESTIGATOR WHO WAS ON SITE. THE LICENSEE WAS FOUND TO BE IN COMPLIANCE WITH NRC REQUIREMENTS WITHIN THE 8 AREAS EXAMINED DURING THE INSPECTION EXCEPT FOR THE FOLLOWING THREE ITEMS: (1) SECURITY ORGANIZATION - FAILURE TO FOLLOW PROCEDURES FOR DOCUMENTING SECURITY INCIDENTS (INFRACTION); (2) PHYSICAL BARRIERS (VITAL AREA) - A VITAL AREA BARRIER DID NOT MEET THE SPECIFICATIONS OF THE LICENSEE'S SECURITY PLAN (INFRACTION); (3) ACCESS CONTROL (VITAL AREA) - FAILURE TO PERFORM VITAL AREA PATROLS AS COMMITTED (INFRACTION).

INSPECTION MARCH 11-14 (80-07): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 13 INSPECTOR-HOURS ON SITE IN THE AREAS OF IE BULLETIN 79-14, "SEISMIC ANALYSIS FOR AS-BUILT SAFETY-RELATED PIPING SYSTEMS" AND LICENSEE ACTION ON PREVIOUS INSPECTION FINDINGS. OF THE TWO AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION MARCH 24-26 (80-09): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 17 INSPECTOR-HOURS ON SITE IN THE AREAS OF IE BULLETIN 79-14, "SEISMIC ANALYSIS FOR AS-BUILT SAFETY-RELATED PIPING SYSTEMS," AND LICENSEE ACTIONS ON PREVIOUS INSPECTION FINDINGS. OF THE TWO AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

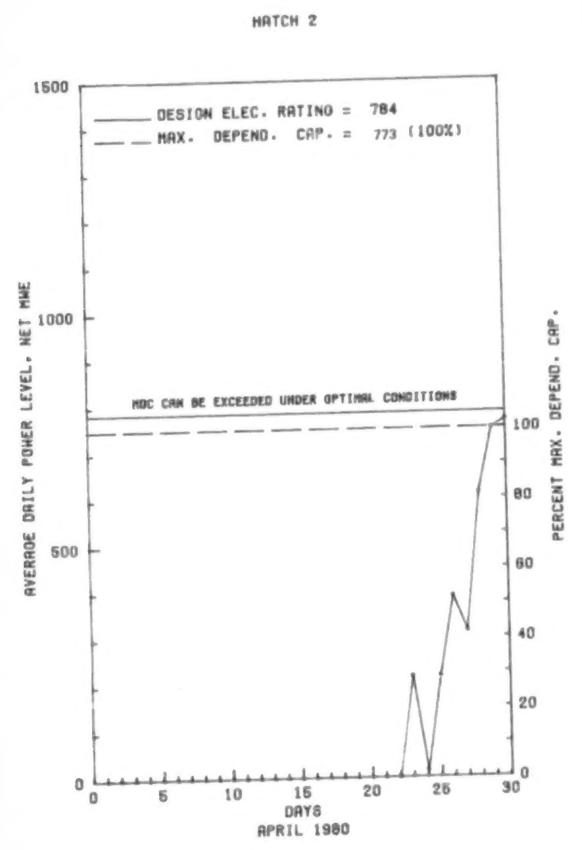
INSPECTION FEBRUARY 2 - MARCH 7 (80-11): THIS INSPECTION INVOLVED 56 INSPECTOR-HOURS ON SITE IN THE AREAS OF TECHNICAL SPECIFICATION COMPLIANCE.

REVIEWED BY OMPA LOEB: Earl Hill **DATE:** 6-12-80

REVIEWED BY NRR DOR: **DATE:** 6-13-80

HATCH 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-366
2. Utility Contact: J. Rodan (912) 367-7781
3. Licensed Thermal Power (MWt): 2436
4. Nameplate Rating (Gross MWe): 1000 x .85 = 850
5. Design Electrical Rating (Net MWe): 784
6. Maximum Dependable Capacity (Gross MWe): 806
7. Maximum Dependable Capacity (Net MWe): 773
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: CHANGED NET MDC TO REFLECT NEW UTILITY RATINGS
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	719.0	2,903.0	5,736.0
12. Hours Reactor Was Critical	226.8	1,613.8	4,277.3
13. Reactor Reserve Shutdown Hours	0	0	0
14. Hours Generator On-Line	164.3	1,508.8	3,923.5
15. Unit Reserve Shutdown Hours	0	0	0
16. Gross Thermal Energy (MWH)	259,210	3,321,790	8,815,852
17. Gross Electrical Energy (MWH)	83,120	1,119,180	2,951,060
18. Net Electrical Energy (MWH)	75,590	1,061,257	2,818,394
19. Unit Service Factor	22.9	52.0	68.4
20. Unit Availability Factor	22.9	52.0	68.4
*21. Unit Capacity Factor (MDC Net)	13.6	47.3	63.6
22. Unit Capacity Factor (DER Net)	13.4	46.6	62.7
23. Unit Forced Outage Rate	14.3	2.8	5.7

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-06	03/01/80	S F	480.0 46.8	H	4		SH	ZZZZZZ	VENT HEADER DEFLECTOR AND 18-MONTH SURVEILLANCE OUTAGE.
80-07	04/23/80	S	.5	B	2		ZZ	TURBIN	UNIT BROUGHT OFF LINE FOR TURBINE OVERSPEED TESTING AND REMAINED DERATED FOR TESTING.
80-08	04/24/80	F	27.4	A	3		EC	ZZZZZZ	RX SCRAM ON TURBINE TRIP DUE TO MOMENTARY LOSS OF DC TO EHC. THE UNIT REMAINED DERATED DUE TO OFFGAS PROBLEMS, PCIOMR AND TO REPAIR STEAM LEAKS.

SUMMARY

THE UNIT CAME ON LINE ON 04/22/80 FOLLOWING AN OUTAGE FOR TORUS VENT HEADER DEFLECTOR INSTALLATION AND 18-MONTH SURVEILLANCE, AND WAS REMOVED FROM SERVICE ON 04/23/80 FOR TURBINE OVERSPEED TESTING. A REACTOR SCRAM FOLLOWING A TURBINE TRIP OCCURRED ON 04/24/80 FROM MOMENTARY LOSS OF DC POWER TO THE ELECTROHYDRAULIC COOLING SYSTEM.

LEGEND

[1] Type:
F-Forced
S-Scheduled

[2] Reason:
A-Equipment Failure
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error
H-Other

[3] Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Continuations
5-Load Reduction
9-Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

REPORTABLE OCCURRENCES, HOUSEKEEPING, OPERATOR PERFORMANCE, OVERALL PLANT OPERATIONS, QUALITY ASSURANCE PRACTICES, STATION AND CORPORATE MANAGEMENT PRACTICES, CORRECTIVE AND PREVENTATIVE MAINTENANCE ACTIVITIES, SITE SECURITY PROCEDURES, RADIATION CONTROL ACTIVITIES AND SURVEILLANCE ACTIVITIES. OF THE ELEVEN AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN SIX AREAS; SIX ITEMS OF NONCOMPLIANCE WERE FOUND IN FIVE AREAS (INFRACTION - FAILURE TO FOLLOW PROCEDURE; INFRACTION - FAILURE TO OPERATE THE RSCS PROPERLY; INFRACTION - FAILURE TO MAKE IMMEDIATE NOTIFICATION TO THE NRC; INFRACTION - FAILURE TO LIMIT ROD MOTION WITH RSCS INOPERABLE; INFRACTION - FAILURE TO EVALUATE PROCEDURE CHANGE; INFRACTION - FAILURE TO PROVIDE UPDATED PLANT DRAWINGS IN THE CONTROL ROOM).

INSPECTION MARCH 10-19 (80-12): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 33 INSPECTOR-HOURS ON SITE IN THE AREAS OF RADIOLOGICAL AND NONRADIOLOGICAL ENVIRONMENTAL PROTECTION INCLUDING MANAGEMENT CONTROLS, REVIEW OF PREVIOUS ENFORCEMENT MATTERS, REVIEW OF ENVIRONMENTAL SURVEILLANCE, REVIEW OF SPECIAL SURVEILLANCE AND STUDY ACTIVITIES, STATUS REVIEW OF LICENSEE EVENT REPORTS, SELECTED SOIL AND GROUNDWATER INDEPENDENT SAMPLING. OF THE SIX AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN FIVE AREAS; ONE ITEM OF NONCOMPLIANCE WAS FOUND IN ONE AREA (DEFICIENCY - ENVIRONMENTAL MONITORING).

INSPECTION MARCH 11-14 AND 25-28 (80-13): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 26 INSPECTOR-HOURS ON SITE IN THE AREAS OF INSERVICE INSPECTION ACTIVITIES (UNIT 2) AND MAINTENANCE WELDING PROGRAM (UNIT 1 AND 2). OF THE 2 AREAS INSPECTED, 2 ITEMS OF NONCOMPLIANCE WERE FOUND IN 2 AREAS (DEFICIENCY - FAILURE TO APPROVE ISI PLAN; AND INFRACTION - FAILURE TO CERTIFY WELDING AND NDE INSPECTORS).

INSPECTION APRIL 2-4 (80-15): THIS SPECIAL, ANNOUNCED INSPECTION INVOLVED 8 INSPECTOR-HOURS ON SITE IN THE AREA OF INSTALLED EQUIPMENT REVIEW WITH RESPECT TO THE IE BULLETIN 79-01B RESPONSE. OF THE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION MARCH 26-28 (80-16): INCLUDED REVIEW OF LICENSEE COMMITMENTS MADE DURING THE JANUARY 25, 1980 ENFORCEMENT MEETING, PHYSICAL BARRIERS (PROTECTED AREA), PHYSICAL BARRIERS (VITAL AREA), LIGHTING, ACCESS CONTROL (IDENTIFICATION, AUTHORIZATION AND BADGING), ACCESS CONTROL (SEARCH), AND COMMUNICATIONS. THE INSPECTION INVOLVED 15 INSPECTOR-HOURS ON SITE BY 2 NRC INSPECTORS. THE INSPECTION WAS BEGUN DURING AN OFFSHIFT PERIOD, 5 INSPECTION HOURS WERE ACCOMPLISHED DURING OFFSHIFT PERIODS. THE LICENSEE WAS FOUND TO BE IN COMPLIANCE WITH NRC REQUIREMENTS WITHIN THE 7 AREAS EXAMINED DURING THE INSPECTION EXCEPT FOR THE FOLLOWING 2 ITEMS: (1) ACCESS CONTROL - FAILURE TO CONTROL ALL POINTS OF PERSONNEL ACCESS INTO VITAL AREAS AND TO VITAL EQUIPMENT (INFRACTION); (2) LIGHTING - FAILURE TO MAINTAIN LIGHTING EQUIPMENT AS COMMITTED TO IN THE SECURITY PLAN (DEFICIENCY); (3) CERTIFICATION OF BULLET RESISTANT STRUCTURES (UNRESOLVED).

INSPECTION MARCH 31 - APRIL 2 (80-18): THIS SPECIAL, ANNOUNCED INSPECTION INVOLVED 16 INSPECTOR-HOURS ON SITE IN THE AREAS OF FOLLOWUP OF RESIDENT-INSPECTOR IDENTIFIED ITEMS OF NONCOMPLIANCE, UNIT STARTUP OPERATIONS AND POST-MODIFICATION TESTING. OF THE THREE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN TWO AREAS; ONE APPARENT ITEM OF NONCOMPLIANCE WAS FOUND IN ONE AREA (INFRACTION - FAILURE TO PERFORM POST-MODIFICATION TESTING).

ENFORCEMENT MEETING APRIL 16 (80-20): DISCUSSED LICENSEE'S ENFORCEMENT HISTORY, OPERATIONAL EVENTS, AND SECURITY ISSUES. GEORGIA POWER COMPANY REPRESENTATIVES ACKNOWLEDGED THE NRC CONCERNS AND INDICATED THAT CORRECTIVE ACTION WOULD BE IMPLEMENTED IN THE AREAS IDENTIFIED.

HUMBOLDT BAY

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	California/Humboldt 4 miles S of Eureka
2.	Electric Reliability Council	Western Systems Coordinating Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	220
5.	Design Electrical Rating (MWE Net)	65
6.	Date of Initial Criticality	February 16, 1963
7.	Date of Commercial Operation	August 1963
8.	Condenser Cooling Method	Circulating sea water
9.	Condenser Cooling Water Source	Pacific Ocean (Humboldt Bay)

Utility & Contractor Information

10.	Licensee	Pacific Gas & Electric Co.
11.	Corporate Address	77 Beale Street San Francisco, California 94106
12.	Corporate Contact	P. A. Crane, Jr. Asst. General Counsel
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	GE
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	V
17.	IE Branch Chief	J. L. Crews
18.	IE Section Chief	D. M. Sternberg
19.	IE Resident Inspector	J. D. Carlson
20.	Licensing Project Manager	V. L. Rooney
21.	Docket Number	50-133
22.	License Number/Date of Issue	DPR-7, 8/28/62
23.	Public Document Room	Humboldt County Library 636 F Street Eureka, California 95501

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

SYSTEMS AND COMPONENT PROBLEMS:

None.

FACILITY ITEMS (PLANS AND PROCEDURES):

The plant has been shut down since July 1976.

Several items remaining to be completed during the outage have been deferred pending resolution of the seismic issues at the plant site. A date for return to operations has not been established.

MANAGERIAL ITEMS:

None

PLANT STATUS:

Plant is shutdown.

LAST IE SITE INSPECTION DATE: 03/24-28/80

INSPECTION REPORT NO: 50-133/80-01

SUMMARY

INSPECTION SUMMARY

+ No inspections conducted in May 1980.

LICENSE STATUS

Order issued 5/21/76 requiring Utility to prove seismic qualification before resuming operation of unit after outage of 7/2/76.

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE			

REVIEWED BY OMPA LOEB: Earl Hill **DATE:** 6/12/80

REVIEWED BY NRR DOR: Dick Clark **DATE:** 06-13-80

GPO 871-867

HUMBOLDT BAY

AVERAGE DAILY POWER LEVEL (MWe)

OPERATING STATUS

1. Docket Number: 50-133
2. Utility Data Prepared By: B. C. Getty (707) 443-2787
3. Licensed Thermal Power (MWt): 220
4. Nameplate Rating (Gross MWe): 77 X 0.85 = 65
5. Design Electrical Rating (Net MWe): 65
6. Maximum Dependable Capacity (Gross MWe): 0
7. Maximum Dependable Capacity (Net MWe): 0
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons: None
9. Power Level To Which Restricted, If Any (Net MWe): None
10. Reasons For Restrictions, If Any:

This M… …ive

11. Hours In Reporting Period
12. Number Of Hours Reactor Was Cri…
13. Reactor Reserve Shutdow…
14. Hours Generato…
15. Unit R…
1… …H)
…rated (MWH)
… Net …gy Generated (MWH)
19. Unit S… …ce Factor
20. Unit Availability Factor
21. Unit Capacity Factor (Using MDC Net) 1/
22. Unit Capacity Factor (Using DER Net) 1/
23. Unit Forced Outage Rate
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): None
25. If Shut Down At End Of Report Period, Estimated Date of Startup: UNKNOWN

NO DECISION ON FUTURE OPERATION
SHUTDOWN INDEFINITELY

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
SEE SUMMARY BELOW									

SUMMARY

THE UNIT WAS SHUTDOWN ON 7/2/76 FOR SEISMIC MODIFICATIONS. THE UNIT REMAINS IN THE COLD SHUTDOWN CONDITION.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

INDIAN POINT 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	New York/Westchester 25 miles N of New York City
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	PWR/fossil superheat
4.	Power Level (MWT)	615
5.	Design Electrical Rating (MWE Net)	265
6.	Date of Initial Criticality	August 2, 1962
7.	Date of Commercial Operation	October 1962
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Hudson River

Utility & Contractor Information

10.	Licensee	Consolidated Edison Company
11.	Corporate Address	of New York, Inc. 4 Irving Place New York, New York
12.	Corporate Contact	W. Cahill, Vice President
13.	Architect/Engineer	Utility
14.	Nuclear Steam Supply System	B&W
15.	Constructor	Utility

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Section Chief	H. B. KISTER
19.	IE Resident Inspector	T. Rebelowski (914-739-9360) FTS: 656-9220
20.	Licensing Project Manager	L. Olshan x27144
21.	Docket Number	50-03
22.	License Number/Date of Issue	DPR-5, 3/26/62
23.	Public Document Room	White Plains Public Library 100 Martine Avenue White Plains, New York 10601

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

```
    I N S P E C T I O N   S T A T U S

    OTHER ITEMS OF INTEREST

+   MANAGERIAL ITEMS:

+   On April 12, 1980, Mr. M. S. Skoezko
    assumed the duties of Security Administrator.

    PLANT STATUS.

    Defueled.

  LAST IE SITE INSPECTION DATE:  7/24-27/79

        INSPECTION REPORT NO:  50-03/79-02

3.  SUMMARY

    INSPECTION SUMMARY

+   None conducted.
```

LICENSE STATUS

The unit produced electricity to the expiration date (October 31, 1974) of delay granted by the AEC for installation of an ECCS system. The ECCS system was not subsequently installed and the unit was defueled on January 30, 1976.

On February 11, 1980, the NRC requested the utility to show cause as to why the operating authority of the license should not be revoked and why the licensee should not submit a plan within 120 days of this order to decommission the unit.

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
NONE				

REVIEWED BY OMPA LOEB: T. C. Cintula **DATE:** 6•12•80

REVIEWED BY NRR DOR: **DATE:** [illegible]

INDIAN POINT 1

AVERAGE DAILY POWER LEVEL (MWe)

OPERATING STATUS

1. Docket Number: 50-003
2. Utility Data Prepared By: L. J. Kawula (914)-694-6000 x209
3. Licensed Thermal Power (MWt): 615
4. Nameplate Rating (Gross MWe): 275
5. Design Electrical Rating (Net MWe): 265
6. Maximum Dependable Capacity (Gross MWe): 0
7. Maximum Dependable Capacity (Net MWe): 0
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report. Give Reasons: None
9. Power Level To Which Restricted, If Any (Net MWe): None
10. Reasons For Restrictions, If Any

This Month Yr to Date

11. Hours In Reporting Period
12. Number Of Hours Reactor Was Critical
13. Reactor Reserve Shutdown Hours
14. Hours Generator On-Line
15. Unit Reserve Shutdown Ho
16. Gross Thermal E
17. Gross
20. Unit A
21. Unit Capacity Factor (U Net) 1/
22. Unit Capacity Factor (Using DER Net) 1/
23. Unit Forced Outage Rate
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): None
25. If Shut Down At End Of Report Period, Estimated Date of Startup: N/A

NO DECISION ON FUTURE OPERATION
SHUTDOWN INDEFINITELY
ALL FUEL UNLOADED.

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
SEE SUMMARY BELOW									

SUMMARY

UNIT SHUTDOWN ON OCTOBER 31, 1974; ALL FUEL HAS BEEN UNLOADED; NO DECISION ON DECOMMISSIONING DATE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

INDIAN POINT 2

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	New York/Westchester 25 miles N of New York City
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2758
5.	Design Electrical Rating (MWE Net)	873
6.	Date of Initial Criticality	May 22, 1973
7.	Date of Commercial Operation	August 1973
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Hudson River

Utility & Contractor Information

10.	Licensee	Consolidated Edison Company of New York, Inc.
11.	Corporate Address	4 Irving Place New York, New York
12.	Corporate Contact	W. J. Cahill, Jr., Vice President
13.	Architect/Engineer	UE&C
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	J. A. Jones/UE&C

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Senior Inspectc	H. B. KISTER
19.	IE Resident Inspector	T. Rebelowski (914-739-9360) FTS: 656-9220
20.	Licensing Project Manager	L. Olshan (X27144)
21.	Docket Number	50-247
22.	License Number/Date of Issue	DPR-26, 10/19/71
23.	Public Document Room	White Plains Public Library 100 Martine Avenue White Plains, New York 10601

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

FACILITY ITEMS (PLANS AND PROCEDURES):

+ An On-Site Safety Review Group has been established to comply with the sixty-day requirements of The Confirmatory Order issued February 11, 1980, Appendix A, part C.6.

+ MANAGERIAL ITEMS:

+ On April 12, 1980 Mr. M. S. Skoszko assumed the duties of Security Administrator.

PLANT STATUS:

+ At 100% Power.

+ LAST IE SITE INSPECTION DATE: 5/12/80

+ INSPECTION REPORT NO: 50-247/80-07

SUMMARY

INSPECTION SUMMARY

+ 50-247/79-25 - Dec 9 - Feb 2: Routine resident inspection of plant operations including shift logs and records; action on previous inspection findings; observation of physical security; turbine rotor cracking problems; review of periodic reports; review of a seismic event; Short Term Lessons Learned verification; and Emergency Planning. The inspection also included a special review of a radioactive waste discrepancy report. The inspection involved 59 onsite hours by the resident inspectors and 4 onsite hours by one region-based radiation specialist. Two items of noncompliance with Department of Transportation regulations were identified.

+ 50-247/80-03 - Mar 19-21: Inspection (16 onsite hours) by the resident reactor inspector of actions taken to comply with the Confirmatory Order issued on February 11, 1980. No items of noncompliance were identified.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

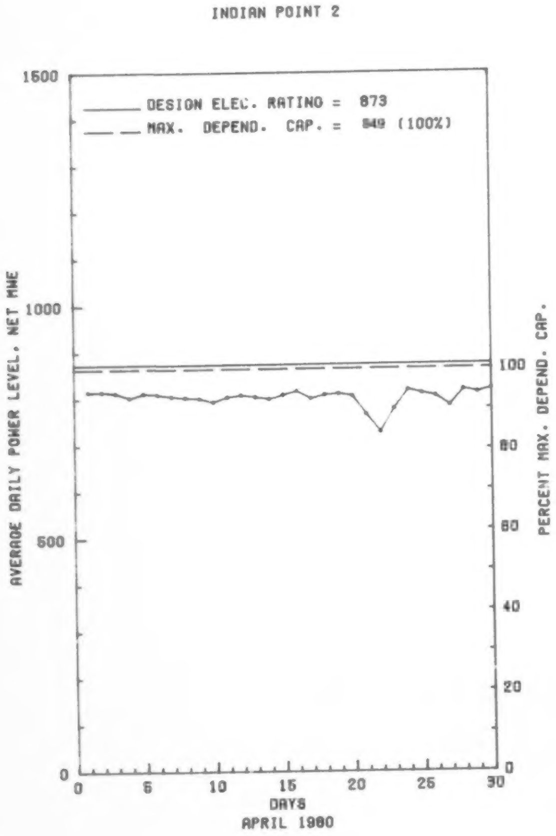
```
==================================================================================
 NUMBER    DATE OF    DATE OF    SUBJECT
            EVENT      REPORT
==================================================================================
 80-001/   02/09/80   03/10/80   Pressure relief valve PCV 1192 leaked excessively
  03L
==================================================================================
```

REVIEWED BY OMPA LOEB: T. C. Cintula **DATE:** 6·12·80

REVIEWED BY NRR DOR: **DATE:**

INDIAN POINT 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

ITEM 21 CALCULATED BY USING A WEIGHTED AVERAGE

1. Docket Number: 50-247
2. Utility Contact: E. EICH (914) 694-6000 @ I.P.
3. Licensed Thermal Power (MWt): 2758
4. Nameplate Rating (Gross MWe): 1126 x 0.9 = 1013
5. Design Electrical Rating (Net MWe): 873
6. Maximum Dependable Capacity (Gross MWe): 885
7. Maximum Dependable Capacity (Net MWe): 849
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: ITEMS 6 & 7 SUMMER RATINGS (4/27/80)
9. Power Level To Which Restricted, If Any (Net MWe): 824
10. Reasons for Restrictions, If Any: REMOVAL OF #3 DISC ON #23 LOW PRESS TURBINE ROTOR.

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	719.0	2,903.0	51,144.0
12. Hours Reactor Was Critical	719.0	2,041.0	34,335.5
13. Reactor Reserve Shutdown Hours	.0	852.6	1,093.8
14. Hours Generator On-Line	719.0	1,985.7	33,374.9
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,948,246	5,154,571	85,987,09:
17. Gross Electrical Energy (MWH)	601,570	1,558,020	26,691,106
18. Net Electrical Energy (MWH)	577,539	1,481,601	25,446,701
19. Unit Service Factor	100.0	68.4	65.3
20. Unit Availability Factor	100.0	68.4	65.3
*21. Unit Capacity Factor (MDC Net)	94.6	60.1	57.7
22. Unit Capacity Factor (DER Net)	92.0	58.5	57.0
23. Unit Forced Outage Rate	.0	9.1	8.2

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	NONE								

SUMMARY

OPERATION WAS AT NEAR FULL POWER DURING THE REPORT PERIOD. LOAD WAS REDUCED ON 04/09/80 TO CHECK FOR CONDENSER TUBE LEAKS, AND ON 04/20/80 DUE TO HIGH VIBRATION IN THE MAIN CIRCULATING WATER PUMP.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

INDIAN POINT 3

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	New York/Westchester 25 miles N of New York City
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	3025
5.	Design Electrical Rating (MWE Net)	965
6.	Date of Initial Criticality	April 6, 1976
7.	Date of Commercial Operation	August 30, 1976
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Hudson River

Utility & Contractor Information

10.	Licensee	Power Authority of the State of New York, Inc.
11.	Corporate Address	10 Columbus Circle New York, New York
12.	Corporate Contact	Peter N. Lyon · Senior Vice President
13.	Architect/Engineer	UE&C
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	WEDCO

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Section Chief	H. B. KISTER
19.	IE Resident Inspector	T. Rebelowski (914-739-9360) FTS 656-9220
20.	Licensing Project Manager	L. Olshan x27144
21.	Docket Number	50-286
22.	License Number/Date of Issue	DPR-64, 12/12/75
23.	Public Document Room	White Plains Public Library 100 Martine Avenue White Plains, New York 10601

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

- There were several trips due to electrical switching problems off site; a load reduction occurred on 4/23 due to turbine noise, circulator and condenser leaks. The unit returned to 100% power on 5/11/80.

FACILITY ITEMS (PLANS AND PROCEDURES):

- An On-Site Safety Review Group has been established to comply with the sixty-day requirements of the Confirmatory Order issued February 11, 1980.

MANAGERIAL ITEMS:

- Mr. George T. Berry has assumed the position of President and Chief Operating Officer. Mr. Peter W. Lyon has assumed the position of Senior Vice President - Nuclear Generation and is now the Corporate Contact.

PLANT STATUS:

- Unit at 100% power.

- LAST IE SITE INSPECTION DATE: 5/12/80
- INSPECTION REPORT NO: 50-286/80-07

SUMMARY

INSPECTION SUMMARY

- 50-286/79-27 - Oct 28 - Nov 24: Routine inspection of plant operations including: log and record reviews; facility tours; action on previous inspection findings; observation of physical security; reported events; radiation protection operations; containment leak rate testing; action related to IE Bulletin 79-14; and, actions on IE Information Notices. The inspection involved 49 onsite hours by two resident inspectors. One item of noncompliance was identified (entering a High Radiation Area).
- 50-286/79-28 - Nov 27: Special unannounced inspection (6 onsite hours) by one region-based inspector to review certain areas of radiation protection and security. Also, the inspector reviewed the licensee's evaluation of a discrepancy in October 1979 personnel dosimetry results. The inspection consisted of the following areas: surveys; procedures and records; training; plant tours; and, liquid effluent discharges. No noncompliances were identified.
- 50-286/79-30 - Dec 10-14: Routine, unannounced inspection (40 onsite hours) by one NRC region-based inspector of the Radioactive Waste System. Also, actions on previous inspection findings and IE Bulletins were reviewed. Areas inspected included: liquid and gaseous radioactive effluent releases; reports of radioactive effluents; effluent release procedures; reactor coolant water quality; and solid radioactive waste. A plant tour of areas was made to observe radioactive waste handling operations. Three noncompliances were identified (failure to perform review, failure to have adequate procedures, failure to label a container of radioactive materials).
- 50-286/80-03 - Mar 19-24: Unannounced inspection by the Resident Inspector (20 onsite hours) of actions taken to comply with the Confirmatory Order issued on February 11, 1980. No items of noncompliance were identified.

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-001/03L	01/29/80	02/28/80	Small leak on nipple to sockolet weld of vent valve C107 in the CVCS system location in containment at the piping penetration
80-002/03L	03/09/80	04/03/80	Failure of redundant heat tracing on pipe between bit tank and the boric acid storage tank
80-003/03L	03/11/80	04/10/80	Auxiliary boiler feed pump system removed from service due to a defective valve body
80-004/03L	03/15/80	04/14/80	Containment spray isolation valve 869B found to be inoperable during surveillance test
80-005/03L	03/16/80	04/17/80	Auxiliary feed pump bearing overheated (AFP No. 31) resulting in a degraded mode operation in accordance with TS 3.4.B

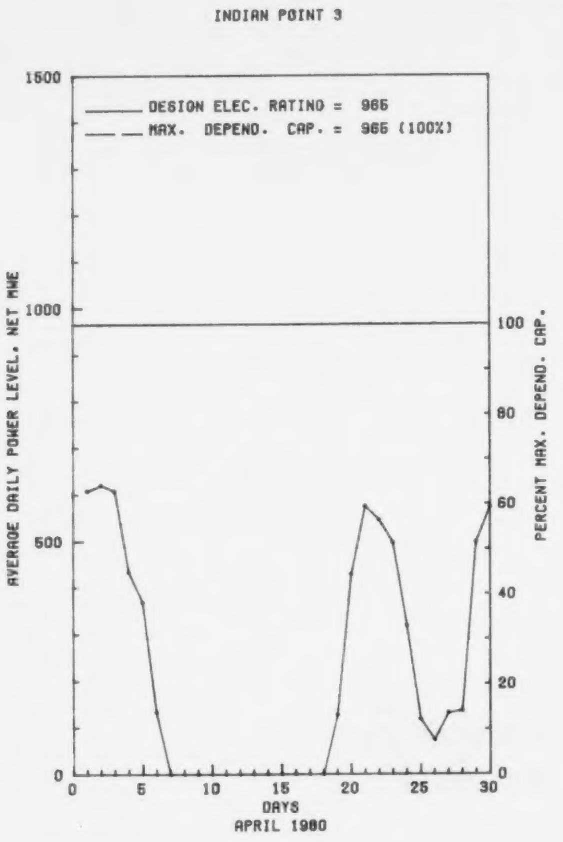
REVIEWED BY OMPA LOEB: T. C. Cintra **DATE:** 6·12·80

REVIEWED BY NRR DOR: ____________ **DATE:** ____________

best copy available

INDIAN POINT 3

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-286
2. Utility Contact: C. Connell (914) 739-8200 x242
3. Licensed Thermal Power (MWt): 3025
4. Nameplate Rating (Gross MWe): 1126 x 0.9 = 1013
5. Design Electrical Rating (Net MWe): 965
6. Maximum Dependable Capacity (Gross MWe): 1000
7. Maximum Dependable Capacity (Net MWe): 965
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): 817
10. Reasons for Restrictions, If Any:
REMOVED TURBINE BLADING ON NO. 31 LP TURBINE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	719.0	2,903.0	32,160.0
12. Hours Reactor Was Critical	529.0	1,622.4	23,207.3
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	408.8	1,234.6	22,267.4
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	695,552	1,986,054	58,090,521
17. Gross Electrical Energy (MWH)	175,450	518,630	18,883,731
18. Net Electrical Energy (MWH)	162,577	478,841	18,122,278
19. Unit Service Factor	56.9	42.5	69.2
20. Unit Availability Factor	56.9	42.5	69.2
21. Unit Capacity Factor (MDC Net)	23.4	17.1	58.4
22. Unit Capacity Factor (DER Net)	23.4	17.1	58.4
23. Unit Forced Outage Rate	43.1	32.1	6.3

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
TURBINE OUTAGE - OCTOBER 80.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
10	04/03/80	F	.0	A	5		HF	PUMPXX	CIRCULATING WATER PUMP BEARINGS MALFUNCTION.
11	04/06/80	F	302.6	A	1		HF	PUMPXX	REPAIR PUMPS, REPLACING BEARINGS.
12	04/22/80	F	.0	A	5		HA	TURBIN	NOISE IN SOUTHERN EXHAUST OF #32 SPINDLE - REDUCED LOAD.
13	04/24/80	F	.0	A	5		HC	HTEXCH	CONDENSER FLANGE AND TUBE LEAKS - REPAIRED FLANGE AND PLUGGED TUBE LEAKS.
14	04/26/80	F	4.4	H	3		EA	RELAYX	345KV FEEDER TRIPPED BECAUSE OF SPURIOUS RELAY ACTUATION.
15	04/26/80	F	3.2	A	3		HA	TURBIN	SPURIOUS INDEPENDENT TURBINE OVERSPEED TRIP - LOOSE CONNECTIONS.

SUMMARY

THE UNIT WAS SHUTDOWN FROM 04/06/80 TO 04/19/80 TO REPAIR BEARINGS IN THE CIRCULATING WATER PUMPS. LOAD REDUCTIONS OCCURRED ON 04/22/80 TO ELIMINATE A RATTLE IN A LOW-PRESSURE TURBINE SPINDLE, AND ON 04/24/80 DUE TO CONDENSER LEAKS. A GENERATOR OUTPUT FEEDER TRIP FROM SPURIOUS RELAY ACTUATION RESULTED IN A UNIT TRIP ON 04/26/80.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

KEWAUNEE

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Wisconsin/Kewaunee 27 miles E of Green Bay
2.	Electric Reliability Council	Mid-America Interpool Network
3.	Type of Reactor	PWR
4.	Power Level (MWT)	1650
5.	Design Electrical Rating (MWE Net)	535
6.	Date of Initial Criticality	March 7, 1974
7.	Date of Commercial Operation	June 1974
8.	Condenser Cooling Method	Once through condenser and return to Lake Michigan
9.	Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10.	Licensee	Wisconsin Public Service Corp.
11.	Corporate Address	P. O. Box 1200 Green Bay, Wisconsin 54305
12.	Corporate Contact	E. R. Mathews Power Supply & Eng.
13.	Architect/Engineer	Fluor Power Services Corp.
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Pioneer Services & Eng. Co.

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	R. F. Warnick
19.	IE Resident Inspector	N. C. Choules
20.	Licensing Project Manager	R. B. Licciardo X28362
21.	Docket Number	50-305
22.	License Number/Date of Issue	DPR-43 issued 12/21/73
23.	Public Document Room	Kewaunee Public Library 822 Juneau Street Kewaunee, Wisconsin 54216

INSPECTION STATUS

ENFORCEMENT STATUS

TECHNICAL SPECIFICATION 6.8.1 STATES, THAT, "WRITTEN PROCEDURES AND ADMINISTRATIVE POLICIES SHALL BE ESTABLISHED, IMPLEMENTED AND MAINTAINED THAT MEET THE REQUIREMENTS AND RECOMMENDATIONS OF SECTION 5.1 AND 5.3 OF ANSI 18.7-1972." SECTION 5.1.6.1 OF ANSI 18.7, 1972, STATES THAT "MAINTENANCE THAT CAN AFFECT THE PERFORMANCE OF SAFETY-RELATED EQUIPMENT SHALL BE PROPERLY PREPLANNED AND PERFORMED IN ACCORDANCE WITH WRITTEN PROCEDURES, DOCUMENTED INSTRUCTIONS OR DRAWINGS APPROPRIATE TO THE CIRCUMSTANCES..." CONTRARY TO THE ABOVE, NO PROCEDURES, INSTRUCTIONS OR DRAWINGS EXISTED AT THE PLANT ON JANUARY 17, 1980, TO INSTRUCT MAINTENANCE PERSONNEL WHERE TO ADD LUBRICATING OIL TO THE EMERGENCY DIESELS WHEN THEY WERE OPERATING OR SHUTDOWN. THIS RESULTED IN THE ADDITION OF FIVE BARRELS OF OIL TO THE 1B EMERGENCY DIESEL'S AIR BOX. IT IS HIGHLY PROBABLE THAT WITH THE ADDITION OF THIS OIL TO THE AIR BOX, THE 1B EMERGENCY DIESEL WAS INOPERABLE FROM ABOUT 10:00 P.M. ON JANUARY 17, 1980, UNTIL THE AFTERNOON OF JANUARY 18, 1980, WHEN THE OIL WAS DRAINED FROM THE AIR BOX.
(8002 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

MANAGERIAL ITEMS

- Effective March 1, 1980, the licensee announced that Mr. D. C. Hintz replaced C. R. Luoma as Plant Manager, and Mr. Luoma was named Assistant Manager of Nuclear Power and is assigned to the Corporate Office. (80-07)

PLANT STATUS

The plant operated routinely at power during the month.

*LAST IE SITE INSPECTION DATE: March 18-21, 1980

- INSPECTION REPORT NO: 80-07

SUMMARY

INSPECTION SUMMARY

- Investigation on February 4-6, (80-03): Because NRC personnel were concerned that NRC was not informed of the condition of the 1B diesel generator during a 24-hour period following a reactor trip and a partial loss of station power which occurred on January 17, 1980, reviewed procedures and records, made observations and interviewed personnel. The investigation involved 31 investigation hours by two investigators. No information was obtained which indicated that licensee personnel withheld information from the NRC concerning the 1B diesel generator. No items of noncompliance were identified.
- Inspection on March 18-21, (80-07): Routine, announced inspection of training, requalification training, calibrations (TS and non-TS), IE Bulletin followup, nonroutine events, design changes, and independent inspection. The inspection involved 55 inspector hours onsite by two NRC inspectors. The inspectors were accompanied by two NRC inspectors in training. No items of noncompliance or deviations were identified.

REPORTS RECEIVED FROM LICENSEE

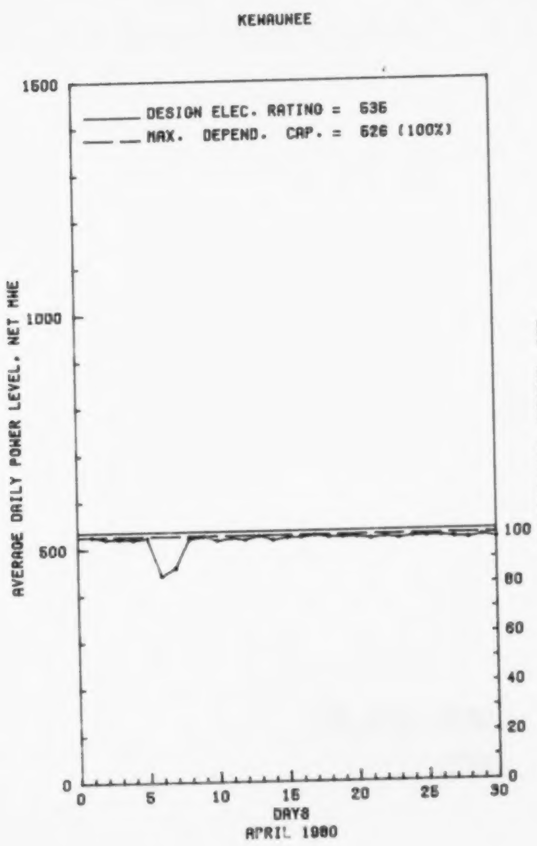
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-14/ 04L-0	03/18/80	04/16/80	Circulation water inlet temperature above 40 degrees F.
80-15/ 03L-0	03/03/80	04/01/80	Containment sump recirculation valve would not open.
80-16/ 03L-0	03/05/80	04/01/80	Diesel generator failed to start.
80-17/ 03L-0	03/10/80	04/09/80	Aux. feedwater pump valve would not close.
80-18/ 03L-0	03/20/80	04/18/80	Transformer disconnect cubicle door was opened and opened the disconnects.

REVIEWED BY OMPA LOEB: **DATE:** 6-12-80

REVIEWED BY NRR DOR: **DATE:** 6.13.80

KEWAUNEE

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-305
2. Utility Contact: G.RUITER (414) 388-2560 X287
3. Licensed Thermal Power (MWt): 1650
4. Nameplate Rating (Gross MWe): 622 x 0.9 = 560
5. Design Electrical Rating (Net MWe): 535
6. Maximum Dependable Capacity (Gross MWe): 545
7. Maximum Dependable Capacity (Net MWe): 526
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

ITEM 21 CALCULATED BY USING A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	719.0	2,903.0	51,506.0
12. Hours Reactor Was Critical	719.0	2,636.0	43,638.5
13. Reactor Reserve Shutdown Hours	.0	152.7	2,330.5
14. Hours Generator On-Line	719.0	2,618.8	42,619.8
15. Unit Reserve Shutdown Hours	.0	.0	10.0
16. Gross Thermal Energy (MWH)	1,175,492	4,236,490	65,387,613
17. Gross Electrical Energy (MWH)	388,700	1,408,900	21,592,700
18. Net Electrical Energy (MWH)	370,626	1,345,182	20,544,260
19. Unit Service Factor	100.0	90.2	82.7
20. Unit Availability Factor	100.0	90.2	82.8
*21. Unit Capacity Factor (MDC Net)	98.0	88.1	78.1
22. Unit Capacity Factor (DER Net)	96.4	86.6	74.6
23. Unit Forced Outage Rate	.0	9.8	5.6

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
REFUELING - MAY 10, 1980 - 6 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	NONE								

SUMMARY

THE UNIT WAS OPERATING FOR THE MONTH OF APRIL WITH NO OUTAGES OR REDUCTIONS.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram.
3--Automatic Scram.
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

LACROSSE

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Wisconsin/Vernon 19 miles S of LaCrosse
2.	Electric Reliability Council	Mid-Continent Area Reliability Coordination Agreement
3.	Type of Reactor	BWR
4.	Power Level (MWT)	165
5.	Design Electrical Rating (MWE Net)	50
6.	Date of Initial Criticality	July 11, 1967
7.	Date of Commercial Operation	November 1, 1969
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Mississippi River

Utility & Contractor Information

10.	Licensee	Dairyland Power Cooperative
11.	Corporate Address	2615 East Avenue South La Crosse, Wisconsin
12.	Corporate Contact	F. Linder
13.	Architect/Engineer	Sargent & Lundy
14.	Nuclear Steam Supply System	Allis-Chalmers
15.	Constructor	Maxson Construction

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. F. HEISHMAN
18.	IE Section Chief	K. R. BAKER
19.	IE Resident Inspector	K. Ridgway
20.	Licensing Project Manager	J. J. Shea
21.	Docket Number	50-409
22.	License Number/Date of Issue	DPR-45 issued 8/28/73
23.	Public Document Room	LaCrosse Public Library 800 Main Street LaCrosse, Wisconsin 54601

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENTS

- Licensee Amendment No. 18, issued February 4, 1980, permits the licensee to expand their fuel storage capability from 134 to 440 fuel assemblies. The new double tier racks are expected to be delivered in early March and installed in May 1980. (80-02)

FACILITY ITEMS

- On February 26, 1980, NRR issued LACBWR a Show Cause Order to show why they should not install a site dewatering system to prevent soil liquefaction in the event of an earthquake. The licensee has 25 days to respond. (80-02)

MANAGERIAL ITEMS

- Dairyland Cooperative has hired an Electrical/Instrument Engineer, an Operations Engineer, two QA Technicians and an Operator since the Management Meeting in October 1979 and is actively pursuing the acquisition of a Nuclear Engineer, Mechanical and Asistant Mechanical Engineer and a Health Physicist. (80-02)

PLANT STATUS

Plant shutdown for equipment modifications per NUREG-0578.

+LAST IE SITE INSPECTION DATE: February 11-15, 1980

+ INSPECTION REPORT NO: 80-02

INSPECTION SUMMARY

- Inspection on February 11-15, (80-02): Routine, unannounced inspection of the licensee's training and retraining programs, surveillance activities; calibration program; procedure changes with regard to Small Break Loss of Coolant Accidents; and followup actions relative to IE Bulletins, IE Circulars, Licensee Event Reports, and special reports. The inspection involved 45 inspector hours onsite by two NRC inspectors. No items of noncompliance were identified in the 10 areas inspected.

REPORTS RECEIVED FROM LICENSEE

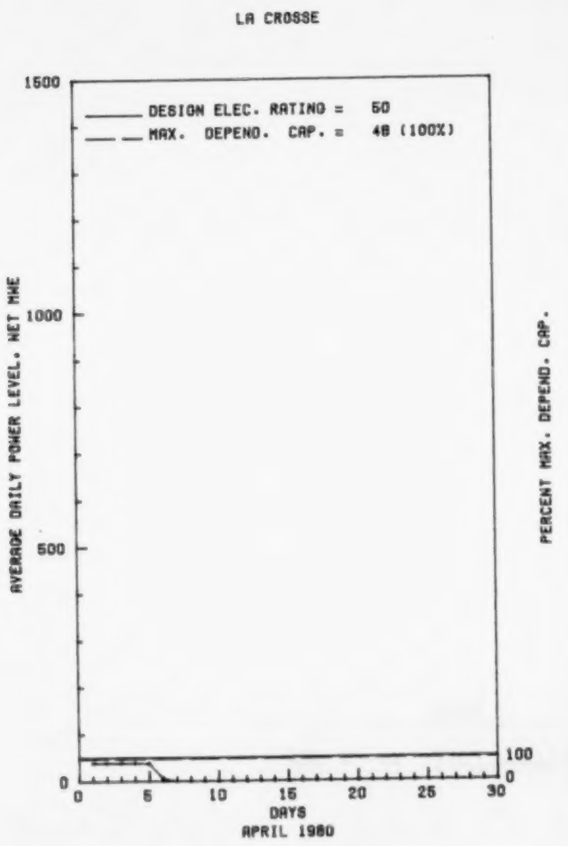
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-01	04/10/80		Electrical Penetrations 3, 4 and 5 failed type B containment leak rate tests.
80-02	04/16/80		Abnormal degradation discovered in the primary reactor containment during periodic type C surveillance leak test.

REVIEWED BY OMPA LOEB: Carl Hill DATE: 6/12/80

REVIEWED BY NRR DOR: JJShea DATE: 6/16/80

LACROSSE

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-409
2. Utility Contact: L. S. Goodman (608) 689-2331
3. Licensed Thermal Power (MWt): 165
4. Nameplate Rating (Gross MWe): 76.8 x 0.85 =65
5. Design Electrical Rating (Net MWe): 50
6. Maximum Dependable Capacity (Gross MWe): 50
7. Maximum Dependable Capacity (Net MWe): 48
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	719.0	2,903.0	92,018.3
12. Hours Reactor Was Critical	212.3	2,338.4	60,164.5
13. Reactor Reserve Shutdown Hours	.0	.0	478.0
14. Hours Generator On-Line	137.4	2,246.9	55,251.9
15. Unit Reserve Shutdown Hours	.0	.0	79.0
16. Gross Thermal Energy (MWH)	18,094	308,884	7,593,663
17. Gross Electrical Energy (MWH)	5,177	91,026	2,290,152
18. Net Electrical Energy (MWH)	4,447	85,291	2,118,026
19. Unit Service Factor	19.1	77.4	60.0
20. Unit Availability Factor	19.1	77.4	60.1
*21. Unit Capacity Factor (MDC Net)	12.9	61.2	48.0
22. Unit Capacity Factor (DER Net)	12.4	58.8	46.0
23. Unit Forced Outage Rate	.0	3.2	6.3

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
REFUELING OUTAGE, AUGUST 18, 1980, 6 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-03	04/06/80	S	581.6	D	1		ZZ	ZZZZZZ	LACBWR WAS SHUTDOWN FOR EQUIPMENT INSTALLATION REQUIRED BY NUREG-0578, INCLUDING POSITION INDICATORS ON RELIEF VALVES AND MANUAL RESETS ON CONTAINMENT ISOLATIONS.

SUMMARY

THE UNIT OPERATED IN AN END-OF-CYCLE COASTDOWN UNTIL 04/06/80, WHEN IT WAS SHUTDOWN FOR EQUIPMENT INSTALLATION REQUIRED BY NUREG-0578. PRIMARY MODIFICATIONS INCLUDED INSTALLATION OF POSITION INDICATORS FOR SAFETY RELIEF VALVES, AND ADDITION OF DIVERSE PARAMETER CLOSURE SIGNALS AND MANUAL RESETS ON SEVERAL CONTAINMENT ISOLATION VALVES. THE UNIT WAS BACK ON LINE AT THE END OF THE REPORT PERIOD.

LEGEND

[1] Type:
F–Forced
S–Scheduled

[2] Reason:
A–Equipment Failure
B–Maintenance or Test
C–Refueling
D–Regulatory Restriction
E–Operator Training & License Examination
F–Administrative
G–Operational Error
H–Other

[3] Method:
1–Manual
2–Manual Scram
3–Automatic Scram
4–Continuations
5–Load Reduction
9–Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

MAINE YANKEE

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Maine/Lincoln 10 miles N of Bath (near Wiscasset)
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2630
5.	Design Electrical Rating (MWE Net)	825
6.	Date of Initial Criticality	October 23, 1972
7.	Date of Commercial Operation	December 28, 1972
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Back River (Atlantic)

Utility & Contractor information

10.	Licensee	Maine Yankee Atomic Power Co.
11.	Corporate Address	20 Turnpike Road Westboro, Massachusetts 01581
12.	Corporate Contact	D. W. Edwards, Assistant to Vice President
13.	Architect/Engineer	Stone & Webster
14.	Nuclear Steam Supply System	Combustion Engineering
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	IE Section Chief	T. T. Martin
19.	IE Resident Inspector	W. J. LAZARUS (X5258)
20.	Licensing Project Manager	C. Nelson
21.	Docket Number	50-309
22.	License Number/Date of Issue	DPR-36, 9/15/72
23.	Public Document Room	Wiscasset Public Library High Street Wiscasset, Maine 04578

INSPECTION STATUS

ENFORCEMENT STATUS

LICENSEE MADE STAFF ORGANIZATIONAL CHANGES W/OUT A PROCEDURE TO DEFINE AUTHORITIES & RESPONSIBILITIES.
(8004 INFRACTION)

LICENSEE PERFORMED CLEANUP OF HIGH CHLORIDES IN RCS W/OUT A PROCEDURE.
(8004 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS:

+ Power escalation continued to full rated power.

+ LAST IE SITE INSPECTION DATE: 4/24/80

+ INSPECTION REPORT NO: 50-309/80-06

SUMMARY

INSPECTION SUMMARY

+ 50-309/80-05 - Mar 17-21: Routine, unannounced inspection of licensee emergency procedures for responding to small-break loss of coolant accidents; followup on Licensee Event Reports (LERs); and review of Monthly Operating Reports. The inspection involved 30 hours on site by one region-based inspector. Noncompliances: None.

REPORTS RECEIVED FROM LICENSEE

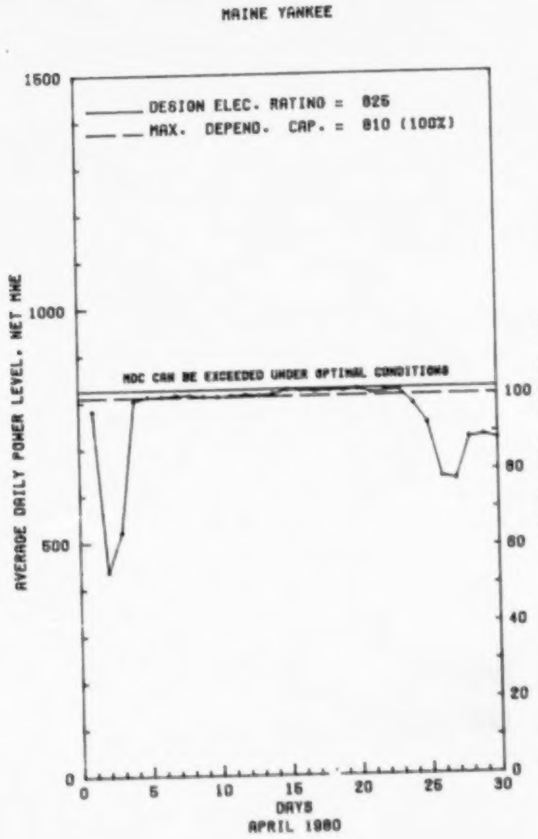
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-006/ 01T	03/12/80	03/25/80	RPS steam generator low level trip inoperative
80-007/ 03L	03/19/80	04/10/80	ESF valves found unlocked/mispositioned
80-008/ 03L	03/21/80	04/14/80	RPS matrix relay failed surveillance test

REVIEWED BY OMPA LOEB: ______________ **DATE:** ______________

REVIEWED BY NRR DOR: Chris Nelson **DATE:** 6-13-80

MAINE YANKEE

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-309
2. Utility Contact: F.C. BEERS (617) 366-9011
3. Licensed Thermal Power (MWt): 2630
4. Nameplate Rating (Gross MWe): 900 x 0.9 = 810
5. Design Electrical Rating (Net MWe): 825
6. Maximum Dependable Capacity (Gross MWe): 850
7. Maximum Dependable Capacity (Net MWe): 810
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

CAPACITY FACTORS CALCULATED WITH A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	719.0	2,903.0	65,531.6
12. Hours Reactor Was Critical	710.9	1,480.5	52,690.4
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	707.7	1,350.1	50,867.8
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,739,186	2,737,275	107,979,338
17. Gross Electrical Energy (MWH)	578,830	901,370	35,474,550
18. Net Electrical Energy (MWH)	552,202	855,275	33,682,461
19. Unit Service Factor	98.4	46.5	77.6
20. Unit Availability Factor	98.4	46.5	77.6
*21. Unit Capacity Factor (MDC Net)	94.8	36.4	66.7
22. Unit Capacity Factor (DER Net)	93.1	35.7	64.5
23. Unit Forced Outage Rate	1.6	4.8	6.8

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	03/29/80	F	86.2	H	5		CC	ZZZZZZ	LOAD REDUCTION TO 20% TO HELP LEACH RESIDUAL CHLORIDES OUT OF THE STEAM GENERATORS.
4-80-5	04/02/80	F	11.3	A	3		IA	INSTRU	A GROUNDED CAPACITOR IN A S/G LEVEL TRANSMITTER LED TO A REACTOR TRIP DURING ROUTINE RPS SURVEILLANCE TESTING.
-	04/24/80	S	157.7	H	5		RC	ZZZZZZ	LOAD REDUCTION TO 75% DUE TO INCREASING CORE D/P. NH4OH WAS ADDED TO HELP REMOVE THE APPARENT BUILDUP OF CRUD ON CORE SURFACES.

SUMMARY

LOAD WAS REDUCED AT THE BEGINNING OF THE REPORT PERIOD TO HELP SOAK OUT CHLORIDES IN THE STEAM GENERATOR, AND ON 04/24/80 WHEN CRUD DEPOSITS ON CORE SURFACES CAUSED AN INCREASE IN D/P. EXCEPT FOR A REACTOR TRIP ON 04/02/80 FROM A GROUNDED CAPACITOR DURING ROUTINE SURVEILLANCE ON THE REACTOR PROTECTION SYSTEM, OPERATION WAS ROUTINE.

LEGEND

[1] Type:
F–Forced
S–Scheduled

[2] Reason:
A–Equipment Failure
B–Maintenance or Test
C–Refueling
D–Regulatory Restriction
E–Operator Training & License Examination
F–Administrative
G–Operational Error
H–Other

[3] Method:
1–Manual
2–Manual Scram.
3–Automatic Scram.
4–Continuations
5–Load Reduction
9–Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

MILLSTONE 1

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Connecticut/New London 5 miles SW of New London
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2011
5.	Design Electrical Rating (MWE Net)	660
6.	Date of Initial Criticality	October 26, 1970
7.	Date of Commercial Operation	March 1971
8.	C¬ndenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Long Island Sound

Utility & Contractor Information

10.	Licensee	Northeast Nuclear Energy Company
11.	Corporate Address	P. O. Box 270 Hartford, Connecticut
12.	Corporate Contact	W. G. Counsil, Vice President
13.	Architect/Engineer	Ebasco
14.	Nuclear Steam Supply System	GE
15.	Constructor	Ebasco

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	IE Section Chief	R. KEIMIG
19.	IE Resident Inspector	J. SHEDLOSKY (203-447-3179) FTS: 642-7287
20.	Licensing Project Manager	J. J. Shea
21.	Docket Number	50-245
22.	License Number/Date of Issue	DPR-21, 10/7/70
23.	Public Document Room	Waterford Public Library Rope Ferry Road Route 156 Waterford, Connecticut 06385

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS:

- The unit operated at a nominal 100% power.
- LAST IE SITE INSPECTION DATE: 4/1-30/80
- INSPECTION REPORT NO: 50-245/80-06

SUMMARY

INSPECTION SUMMARY

- 50-245/80-02 - Jan 2-4, 9-11, 29-31 & Feb 13-15, 20-22: Special announced investigation conducted by three region-based investigators of Isolation Condenser System piping, pipe support, and dry-well concrete for their ability to withstand loads imposed by seismic, blowdown, or water-hammer event. The investigators reviewed stress reports of piping and pipe supports, witnessed tests and NDE examination of the piping and supports, visually examined cracked concrete, interviewed engineering and technical personnel, and conducted a technical evaluation of data and other information presented by the licensee to determine the structural adequacy of this system. The investigation involved 174 investigator hours by three region-based investigators, 34 hours by a NRR engineer and 6 hours by a region-based section chief at the licensee's corporate offices, the licensee's contractor's offices and the plant site. Noncompliances: One (failure to promptly identify, evaluate, and document a nonconformance).
- 50-245/80-04 - Mar 1-29: Routine, onsite, regular and backshift inspection by two resident inspectors (84 hours). Areas inspected included the control room and the accessible portions of the reactor, turbine, radioactive waste, gas turbine generator, and intake buildings; radiation protection; physical security; fire protection; plant operating records; surveillance testing; calibration; maintenance; core power distribution limits; and reporting to the NRC. Noncompliances: None.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

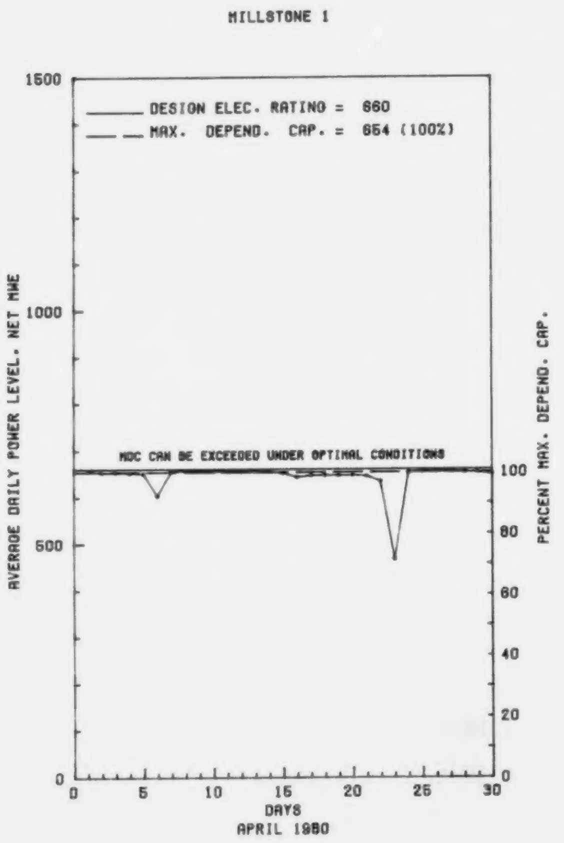
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
NONE				

REVIEWED BY OMPA LOEB: ______ DATE: ______

REVIEWED BY NRR DOR: J. J. Shea DATE: 6/16/80

MILLSTONE 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-245
2. Utility Contact: R. Young (203) 447-1792 x475
3. Licensed Thermal Power (MWt): 2011
4. Nameplate Rating (Gross MWe): 735 x 0.9 = 662
5. Design Electrical Rating (Net MWe): 660
6. Maximum Dependable Capacity (Gross MWe): 684
7. Maximum Dependable Capacity (Net MWe): 654
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	719.0	2,903.0	82,583.0
12. Hours Reactor Was Critical	719.0	2,903.0	63,288.0
13. Reactor Reserve Shutdown Hours	.0	.0	935.0
14. Hours Generator On-Line	719.0	2,903.0	60,998.9
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,413,400	4,677,583	108,725,528
17. Gross Electrical Energy (MWH)	484,000	1,583,300	36,814,696
18. Net Electrical Energy (MWH)	463,061	1,511,437	35,151,219
19. Unit Service Factor	100.0	100.0	73.9
20. Unit Availability Factor	100.0	100.0	73.9
21. Unit Capacity Factor (MDC Net)	98.5	79.6	65.1
22. Unit Capacity Factor (DER Net)	97.6	78.9	64.5
23. Unit Forced Outage Rate	.0	.0	15.9

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUEL & MAINT. - 09/19/80 - 8 TO 10 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
02	04/23/80	F	.0	A	5		HB	PIPEXX	UNIT POWER WAS REDUCED TO PLUG LEAKING MAIN CONDENSER TUBES. UNIT POWER WAS INCREASED AFTER LEAKING TUBES WERE PLUGGED.

SUMMARY

OPERATION WAS AT NEAR FULL POWER DURING THE REPORT PERIOD, EXCEPT FOR A LOAD REDUCTION ON 4/6/80 FOR TURBINE STOP VALVE TESTING AND REBRUSHING A RECIRCULATION M/G SET AND ON 4/23/80 TO PLUG LEAKING MAIN CONDENSER TUBES.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

MILLSTONE 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Connecticut/New London 5 miles SW of New London
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2700
5.	Design Electrical Rating (MWE Net)	870
6.	Date of Initial Criticality	October 17, 1975
7.	Date of Commercial Operation	December 26, 1975
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Long Island Sound

Utility & Contractor Information

10.	Licensee	Northeast Nuclear Energy Company
11.	Corporate Address	P. O. Box 270 Hartford, Connecticut
12.	Corporate Contact	W. G. Counsil, Vice President
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Combustion Engineering
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	IE Section Chief	E. McCabe
19.	IE Resident Inspector	J. T. SHEDLOSKY (203-447-3179) R. Zimmerman FTS: 642-7287
20.	Licensing Project Manager	E. L. Conner, Jr.
21.	Docket Number	50-336
22.	License Number/Date of Issue	DPR-65, 8/1/75
23.	Public Document Room	Waterford Public Library Rope Ferry Road Route 156 Waterford, Connecticut 06385

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

- Pressurizer spray valve stuck open resulting in a reactor trip on April 29. #1 feedwater regulating valve malfunctioned, due to a scored air cylinder, and resulted in a reactor trip on April 30.

PLANT STATUS:

- The plant operated at a nominal 100% power, with the exception of reactor trips on April 29 and April 30.

- LAST IE SITE INSPECTION DATE: 4/1-5/3/80
- INSPECTION REPORT NO: 50-336/80-04

SUMMARY

INSPECTION SUMMARY

- 50-336/80-03 - Mar 1-29: Routine, onsite, regular and backshift inspection by two resident inspectors (84 hours). Areas inspected included the control room and the accessible portions of the enclosure, auxiliary, turbine and intake buildings, and the condensate polishing facility; surveillance testing; calibration; maintenance; core power distribution limits; and reporting to the NRC. Noncompliances: None.

REPORTS RECEIVED FROM LICENSEE

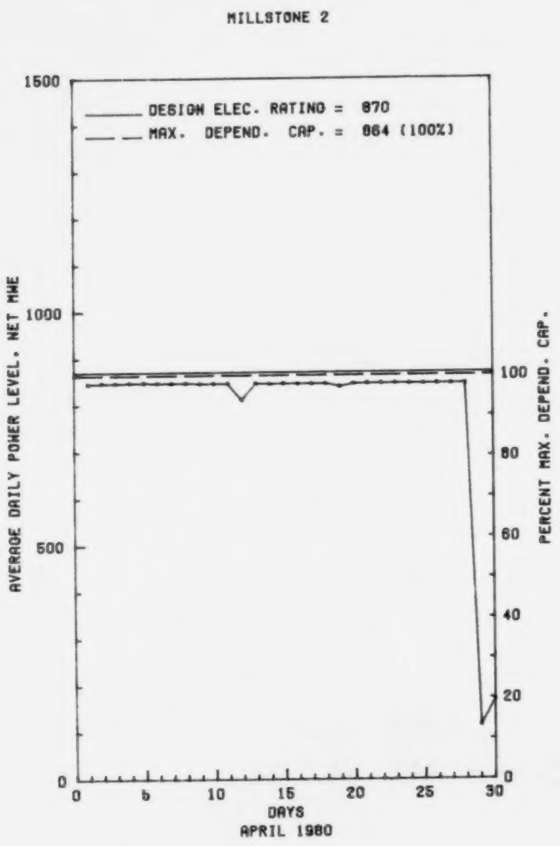
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-005/01T	03/11/80	03/21/80	Incorrect assumption for boron dilution event. Licensee will insure 2 percent S/D margin vice 1 percent to insure adequate margin
80-006/03L	03/10/80	03/27/80	C Charging pump placed out of service to replace packing while "B" pump inoperable
80-007/03L	02/27/80	03/27/80	Condensate storage tank level below TS requirements during plant startup
80-008/03L	03/03/80	04/02/80	Plant computer out of service for 13 hours and 10 minutes. Power was reduced to 90 percent.
80-009/03L	03/21/80	04/10/80	Excessive packing gland leakage from "A" aux feedwater pump. Repaired and returned to service in 4 hours
80-010/03L	03/10/80	04/09/80	One CEA position indication inoperable due to faulty connector/cable assembly
80-011/03L	03/13/80	04/11/80	Hanger bolts on main steam line to the high pressure turbine failed due to fatigue cracking and stress caused by plant trip
80-012/03L	03/14/80	04/11/80	Defective snubber found on main steam line to H.P. turbine

REVIEWED BY OMPA LOEB: ______________ DATE: ______________

REVIEWED BY NRR DOR: ______________ DATE: ______________

MILLSTONE 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-336
2. Utility Contact: G. H. Howlett (203) 447-1791 x364
3. Licensed Thermal Power (MWt): 2700
4. Nameplate Rating (Gross MWe): 1011 x 0.9 = 910
5. Design Electrical Rating (Net MWe): 870
6. Maximum Dependable Capacity (Gross MWe): 895
7. Maximum Dependable Capacity (Net MWe): 864
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

CAPACITY FACTORS CALCULATED WITH A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	719.0	2,903.0	38,111.0
12. Hours Reactor Was Critical	700.5	2,813.1	27,891.8
13. Reactor Reserve Shutdown Hours	.0	.0	2,072.4
14. Hours Generator On-Line	685.1	2,744.2	26,564.4
15. Unit Reserve Shutdown Hours	.0	.0	335.4
16. Gross Thermal Energy (MWH)	1,831,557	7,257,501	64,956,129
17. Gross Electrical Energy (MWH)	595,346	2,373,815	20,958,460
18. Net Electrical Energy (MWH)	573,523	2,284,416	20,064,674
19. Unit Service Factor	95.3	94.5	69.7
20. Unit Availability Factor	95.3	94.5	70.6
*21. Unit Capacity Factor (MDC Net)	92.3	91.1	64.2
22. Unit Capacity Factor (DER Net)	91.7	90.5	62.8
23. Unit Forced Outage Rate	4.7	5.5	22.2

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
REFUELING (CYCLE 4), JULY 1, 1980, 9 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWN/REDUCTIONS

No.	Date	Type^1	Duration (Hours)	Reason^2	Method of Shutting Down Reactor^3	Licensee Event Report No.	System Code^4	Component Code^5	Cause & Corrective Action to Prevent Recurrence
7	04/29/80	F	25.5	A	3		CB	VALVEX	PRESSURIZER SPRAY VALVE STUCK OPEN, RESULTING IN A R.C.S. PRESSURE DROP. SUBSEQUENTLY REACTOR POWER WAS REDUCED, DURING WHICH FEED REGULATING VALVE (2-FW-51A) LOCKED UP (CLOSED). THE REACTOR TRIPPED ON LOW STEAM GENERATOR LEVEL.
8	04/30/80	F	8.4	A	3		HH	VALVEX	FEED REGULATING VALVE (2-FW-51A) LOCKED-UP (CLOSED), RESULTING IN A REACTOR TRIP ON LOW STEAM GENERATOR LEVEL.

SUMMARY

OPERATION WAS ROUTINE DURING THE REPORT PERIOD, EXCEPT FOR REACTOR TRIPS ON 04/29/80 AND 04/30/80 WHEN LOCK-UP AT THE FEED REGULATING VALVE RESULTED IN LOW STEAM GENERATOR LEVEL.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

MONTICELLO

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Minnesota/Wright 30 miles NW of Minneapolis
2.	Electric Reliability Council	Mid-Continent Area Reliability Coordination Agreement
3.	Type of Reactor	BWR
4.	Power Level (MWT)	1670
5.	Design Electrical Rating (MWE Net)	545
6.	Date of Initial Criticality	December 10, 1970
7.	Date of Commercial Operation	June 30, 1971
8.	Condenser Cooling Method	Once Through with Cooling Tower
9.	Condenser Cooling Water Source	Mississippi River

Utility & Contractor Information

10.	Licensee	Northern States Power Co.
11.	Corporate Address	414 Nicollet Mall - 8th Floor Minneapolis, Minnesota 55401
12.	Corporate Contact	G. H. Neils (Oper.) L. O. Mayer (Lic.)
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	GE
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. F. HEISHMAN
18.	IE Section Chief	R. F. Warnick
19.	IE Resident Inspector	C. Brown
20.	Licensing Project Manager	T. Kevern
21.	Docket Number	50-263
22.	License Number/Date of Issue	DPR-22 issued 9/8/70
23.	Public Document Room	Environmental Conservation Library Minneapolis Public Library 300 Nicollet Mall Minneapolis, Minnesota 55401

INSPECTION STATUS

ENFORCEMENT STATUS

A VITAL AREA BARRIER HAD AN OPENING LARGE ENOUGH FOR A PERSON TO CRAWL THROUGH.
(8003 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

FACILITY ITEMS

+ The plant shutdown on February 22, 1980 for a five week refueling outage. (80-02)

PLANT STATUS

+ The plant is currently shutdown due to a drywell to torus leak through the vacuum breakers and to correct a seal problem on the No. 12 recirculation pump.

\+LAST IE SITE INSPECTION DATE: February 26-27 and March 10, 1980

+ INSPECTION REPORT NO: 80-05

SUMMARY

INSPECTION SUMMARY

+ Inspection on February 19-22 and March 3-4, (80-02): Routine unannounced inspection of maintenance, nonroutine event followup; IE Bulletin and Circular followup, calibrations, preparations for refueling, refueling activities, refueling maintenance, and independent inspection. The inspection involved 42 inspector hours onsite by one NRC inspector. The inspector was accompanied by one NRC inspector trainee. No items of noncompliance or deviations were identified.

+ Inspection on February 26-27 and March 10, (80-05): Inservice Inspection (ISI) and torus structural modifications, QA/QC programs, implementing procedures, work activities, and review and evaluation of nondestructive examination (NDE) and welding data and reports. The inspection involved 17 inspector hours onsite by one NRC inspector. No items of noncompliance or deviations were identified.

REPORTS RECEIVED FROM LICENSEE

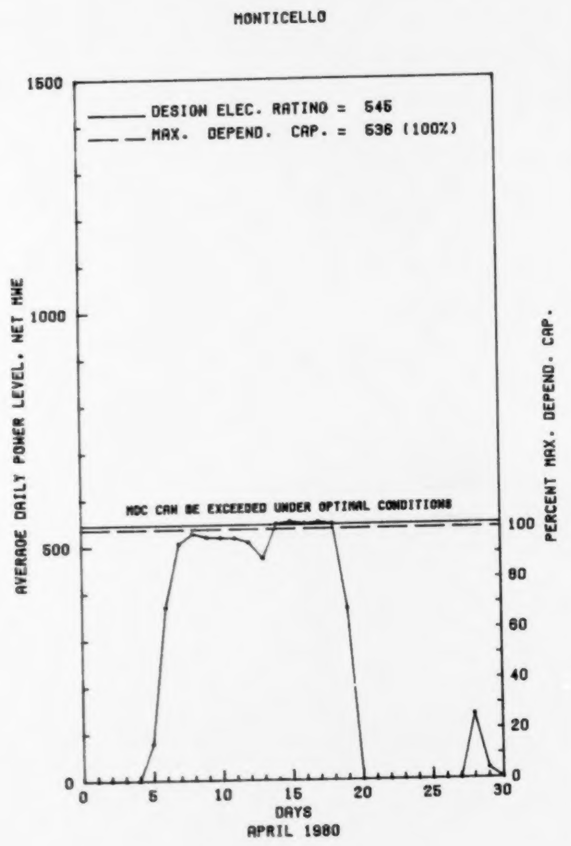
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-02/ 01T-0	02/25/80	04/08/80	Failure of nine CIV's to pass local leak rate tests.
80-03/ 01T-0	02/25/80	04/08/80	Failure of CIV to pass local leak rate tests.
80-04/ 01T-0	02/23/80	04/08/80	Failure of CIV to pass local leak rate tests.
80-05/ 01T-0	03/03/80	04/08/80	Failure of CIV to pass local leak rate tests.
80-08/ 01T-0	03/03/80	04/08/80	Failure of CIV to pass local leak rate tests.
80-09/ 01T-0	03/06/80	04/08/80	Failure of CIV to pass local leak rate tests.
80-10/ 01T-0	03/24/80	04/08/80	Failure of CIV to pass local leak rate tests.
80-12/ 01T-0	03/07/80	04/08/80	Failure of CIV to pass local leak rate tests.
80-14/ 01T-0	03/14/80	04/08/80	Failure of CIV to pass local leak rate tests.
80-15/ 01T-0	03/17/80	03/27/80	Unqualified splices found in MSIV control cables.
80-16/ 01T-0	03/27/80	04/10/80	Safety relief valve air line seismic problems.
80-17/ 01T-0	03/28/80	04/11/80	Reduction of CRD Hydraulic piping frame support design margins caused by EDS computer code error.

REVIEWED BY OMPA LOEB: ____________ DATE: ____________

REVIEWED BY NRR DOR: T. A. Kevern DATE: 6/13/80

MONTICELLO

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-263
2. Utility Contact: A. L. Myrabo (612) 295-5151
3. Licensed Thermal Power (MWt): 1670
4. Nameplate Rating (Gross MWe): 632 x 0.9 = 569
5. Design Electrical Rating (Net MWe): 545
6. Maximum Dependable Capacity (Gross MWe): 564
7. Maximum Dependable Capacity (Net MWe): 536
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	719.0	2,903.0	77,448.0
12. Hours Reactor Was Critical	416.5	1,640.1	63,171.0
13. Reactor Reserve Shutdown Hours	.0	.0	940.7
14. Hours Generator On-Line	361.6	1,572.7	61,542.8
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	536,251	2,090,129	101,032,991
17. Gross Electrical Energy (MWH)	182,400	703,550	31,681,610
18. Net Electrical Energy (MWH)	173,898	668,740	30,300,944
19. Unit Service Factor	50.3	54.2	79.5
20. Unit Availability Factor	50.3	54.2	79.5
21. Unit Capacity Factor (MDC Net)	45.1	43.0	73.0
22. Unit Capacity Factor (DER Net)	44.4	42.3	71.8
23. Unit Forced Outage Rate	35.2	11.5	6.0

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
MARCH 28, 1981 - REFUELING OUTAGE - 56 DAYS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: 05/16/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
4	02/22/80	S	101.6	C	4		ZZ	ZZZZZZ	CONTINUATION OF 1980 REFUELING OUTAGE.
5	04/05/80	S	3.2	B	1		HA	TURBIN	TURBINE OVERSPEED TEST.
6	04/19/80	F	151.2	A	3		CB	PUMPXX	TURBINE TRIP ON HIGH LEVEL WHILE REDUCING POWER AFTER RECIRC PUMP SEAL FAILURE. SEALS ON BOTH RECIRC PUMPS REPLACED.
7	04/26/80	S	57.5	A	9		CB	PUMPXX	OUTAGE TAKEN TO REPAIR RECIRC PUMP SEAL.
8	04/29/80	F	43.8	A	1		CB	PUMPXX	PLANT SHUTDOWN TO REPAIR RECIRC PUMP SEAL.

SUMMARY

THE UNIT WAS OFF LINE UNTIL 04/05/80 BECAUSE OF REFUELING. OUTAGES WERE NECESSARY FROM 04/19/80 THROUGH 04/27/80, AND FROM 04/29/80 THROUGH THE END OF THE PERIOD, TO REPAIR RECIRCULATION PUMP SEALS.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

NINE MILE POINT 1

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	New York/Oswego 8 miles NE of Oswego
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	1850
5.	Design Electrical Rating (MWE Net)	620
6.	Date of Initial Criticality	September 5, 1969
7.	Date of Commercial Operation	December 1969
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Lake Ontario

Utility & Contractor Information

10.	Licensee	Niagara Mohawk Power Corporation
11.	Corporate Address	300 Erie Boulevard West Syracuse, New York 13202
12.	Corporate Contact	R. Schneider, Vice President Electric Operations
13.	Architect/Engineer	Utility
14.	Nuclear Steam Supply System	GE
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Section Chief	H. B. KISTER
19.	IE Resident Inspector	W. BAUNACK (X5253)
20.	Licensing Project Manager	P. Polk
21.	Docket Number	50-220
22.	License Number/Date of Issue	DPR-63, 12/26/74
23.	Public Document Room	State University College of Oswego Penfield Library - Documents Oswego, NY 13126 (315)341-2323

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

```
INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS:

+    The unit operated continuously at essentially
     full power with the exception of a two day power
     reduction to approximately 60% power due to an
     attached feedwater pump lube oil fire which occurred
     on 4/21.  Representative stack and offgas release
     rates are 19.6 and 9800 microcuries per second,
     respectively.

+ LAST IE SITE INSPECTION DATE:  5/9/80

+         INSPECTION REPORT NO:  50-220/80-04

     SUMMARY

     INSPECTION SUMMARY

+    50-220/80-02 - Mar 3-7:  Routine, unannounced inspection by
     two region-based inspectors of small break loss of coolant
     accident procedures; small break loss of coolant procedures
     training; verification of as built safety-related piping
     systems; isolation condenser piping supports; and facility
     tours.  Arrangements for establishment of an onsite office
     for a full-time NRC inspector were also made.  The inspection
     involved 64 inspector-hours onsite by two NRC region-based
     inspectors and one supervisor.  Noncompliances: None.
```

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT

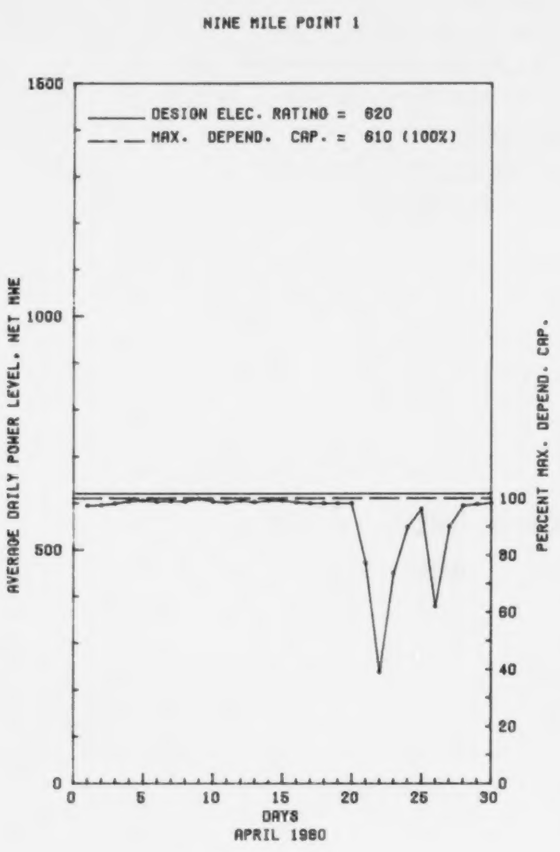
NONE

REVIEWED BY OMPA LOEB: ________ **DATE:** ________

REVIEWED BY NRR DOR: Philip J. Polk **DATE:** 6/6/80

NINE MILE POINT 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-220
2. Utility Contact: B. Hawksley (315) 343-2110 x1,12
3. Licensed Thermal Power (MWt): 1850
4. Nameplate Rating (Gross MWe): 755 x 0.85 = 642
5. Design Electrical Rating (Net MWe): 620
6. Maximum Dependable Capacity (Gross MWe): 630
7. Maximum Dependable Capacity (Net MWe): 610
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe) NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	719.0	2,903.0	97,015.0
12. Hours Reactor Was Critical	719.0	2,724.4	68,677.1
13. Reactor Reserve Shutdown Hours	.0	.0	1,204.2
14. Hours Generator On-Line	719.0	2,694.3	65,504.8
15. Unit Reserve Shutdown Hours	.0	.0	20.2
16. Gross Thermal Energy (MWH)	1,255,271	4,636,727	106,378,126
17. Gross Electrical Energy (MWH)	421,776	1,560,989	35,070,499
18. Net Electrical Energy (MWH)	408,418	1,509,972	33,959,737
19. Unit Service Factor	100.0	92.8	71.2
20. Unit Availability Factor	100.0	92.8	71.2
21. Unit Capacity Factor (MDC Net)	93.1	85.3	60.5
22. Unit Capacity Factor (DER Net)	91.6	83.9	59.5
23 Unit Forced Outage Rate	.0	.0	9.0

24 Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE

25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-12	04/21/80	F	.0	A	5				LOAD REDUCTION TO 60% POWER DUE TO LOSS OF SHAFT DRIVEN FW PUMP (FIRE).
80-13	04/26/80	S	.0	B	5				LOAD REDUCTION TO 40% POWER TO INSPECT CONDENSER TUBES AND PLUG LEAKS.

SUMMARY

OPERATION WAS ROUTINE DURING THE REPORT PERIOD WITH THE NO. 15 REACTOR RECIRCULATION PUMP OUT OF SERVICE AND ISOLATED DUE TO MECHANICAL PROBLEMS. LOAD WAS REDUCED FROM 04/21/80 THROUGH 04/22/80 FOLLOWING A FIRE ON A SHAFT DRIVEN FEEDWATER PUMP'S PIPING INSULATION. A LOAD REDUCTION ALSO OCCURRED ON 04/26/80 FOR MAINTENANCE ON THE MAIN CONDENSER WATERBOXES.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

NORTH ANNA 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Louisa County, Virginia 40 Miles NW of Richmond, Virginia
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	PWR
4.	Licenses Power Level (MWT)	2775
5.	Design Electrical Rating (MWE Net)	907
6.	Date of Initial Criticality	April 5, 1978
7.	Date of Commercial Operation	June 6, 1978
8.	Condenser Cooling Method	Once through
9.	Condenser Cooling Water Source	Lake Anna

Utility & Contractor Information

10.	Licensee	Virginia Electric & Power Co.
11.	Corporate Address	P. O. Box 26666 Richmond, Virginia 23261
12.	Corporate Contact	J. H. Ferguson Exec. V.P. - Power
13.	Architect/Engineer	Stone & Webster
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. Lewis
18.	IE Section Chief	P. J. Kellog
19.	IE Resident Inspector	M. S. Kidd (703) 894-6927
20.	Licensing Project Manager	L. Engle
21.	Docket Number	50-338
22.	License Number/Date of Issue	NPF-4, 11/26/77
23.	Public Document Room	Alderman Library/Manuscripts Dept. Univ. of Virginia/Charlottesville VA 22901 & Louisa County Courthouse, Louisa, VA 23093

INSPECTION STATUS

ENFORCEMENT STATUS

AS REQUIRED BY UNIT 1 TECHNICAL SPECIFICATION 6.5.1.6.B, THE STATION NUCLEAR SAFETY AND OPERATING COMITTEE (SNSOC) SHALL REVIEW ALL PROPOSED TESTS AND EXPERIMENTS WHICH AFFECT NUCLEAR SAFETY. CONTRARY TO THE ABOVE, A TEST INVOLVING "PERKING" OF A MAIN STEAM LINE NON-RETURN VALVE TO DETERMINE IF THIS WAS THE SOURCE OF LOOSE PARTS INDICATIONS ON FEBRUARY 23, 1980, WAS NOT REVIEWED BY THE SNSOC PRIOR TO CONDUCT. THE TEST RESULTED IN A SAFETY INJECTION ACTUATION.
(8011 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

UNIT IS OPERATING AT 100% POWER.

♦ LAST IE SITE INSPECTION DATE: APRIL 25, 1980

♦ INSPECTION REPORT NO: 50-338/80-20

SUMMARY

INSPECTION SUMMARY

INSPECTION MARCH 10-14 (80-12): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 15 INSPECTOR-HOURS ON SITE IN THE AREAS OF POST-REFUELING OPERATIONS, START-UP TESTING, CORE PERFORMANCE MONITORING AND FUEL HANDLING. IN THE FOUR AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION MARCH 24-27 (80-13): THIS ROUTINE UNANNOUNCED INSPECTION INVOLVED 22 INSPECTOR-HOURS ON SITE IN THE AREAS OF LICENSEE EVENT REPORTS AND PREVIOUS INSPECTION FINDINGS. OF THE TWO AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION MARCH 19-20 (80-15): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 11 INSPECTOR-HOURS ON SITE IN THE AREAS OF DEGRADED GRID VOLTAGE PROTECTION AND FOLLOW-UP INSPECTION OF LICENSEE IDENTIFIED ITEMS. OF THE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

♦ INSPECTION MARCH 3 - APRIL 11 (80-16): THIS ROUTINE INSPECTION BY THE RESIDENT INSPECTOR INVOLVED SIX HOURS ON SITE IN THE AREAS OF LICENSEE EVENT REPORTS AND SELECTED SHORT TERM LESSONS LEARNED (NUREG-0578) REQUIREMENTS. NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

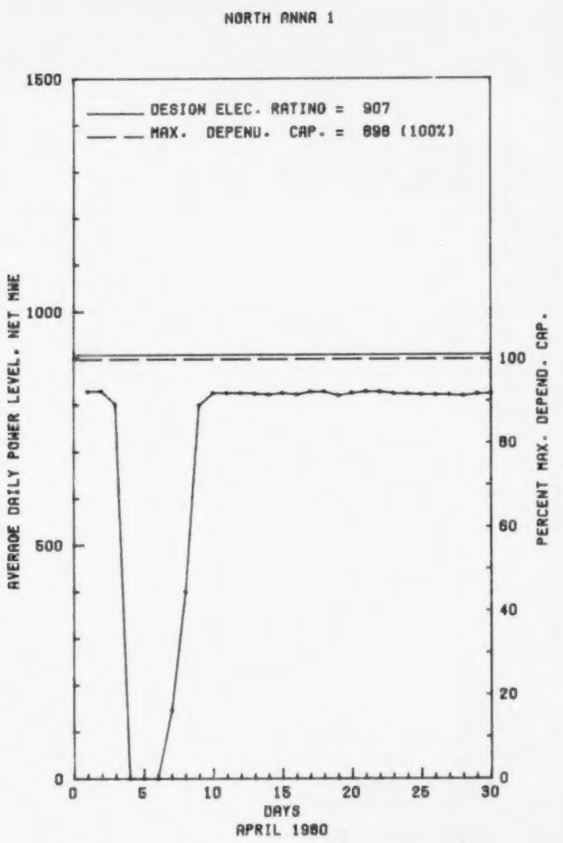
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
80-25/ 03L-0	02/15/80	03/14/80	30 DAY LER	STEAMLINE PRESSURE PROTECTION CHANNEL TRIP SETPOINT DRIFT.
80-26/ 03L-0	02/17/80	03/18/80	30 DAY LER	RECIRCULATION SPRAY PUMP CASING COOLING VALVE FAILED TO OPEN DURING ROUTINE SURVEILLANCE.
80-29/ 03L-0	02/26/80	03/27/80	30 DAY LER	RADIOACTIVE LIQUID LEAK INTO AN UNCONTROLLED AREA FROM A TEMPORARY FITTING.
80-30/ 03L-0	02/27/80	03/28/80	30 DAY LER	GASEOUS WASTE CHARCOAL FILTER SURVEILLANCE NOT CONDUCTED.
80-31/ 03L-0	02/29/80	03/28/80	30 DAY LER	CASING COOLING TANK LEVEL TRANSMITTER NOT SEISMICALLY QUALIFIED.
80-32/ 03L-0	02/15/80	03/14/80	30 DAY LER	EMERGENCY DIESEL 1H TRIPPED ON OVERSPEED ON SIX OCCASIONS DURING TESTING.
80-34/ 01T-0	03/07/80	03/21/80	14 DAY LER	EMERGENCY CORE COOLING SYSTEM PIPING NOT STRESS ANALYZED FOR TEMPERATURES BELOW 70 DEGREES FAHRENHEIT.

REVIEWED BY OMPA LOEB: T.C. Cintola **DATE:** 6·12·80

REVIEWED BY NRR DOR: LBEngle **DATE:** 6.13.80

NORTH ANNA 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-338
2. Utility Contact: W. R. Madison (703) 894-5151
3. Licensed Thermal Power (MWt): 2775
4. Nameplate Rating (Gross MWe): 1089 x 0.9 = 980
5. Design Electrical Rating (Net MWe): 907
6. Maximum Dependable Capacity (Gross MWe): 928
7. Maximum Dependable Capacity (Net MWe): 898
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	719.0	2,903.0	16,680.0
12. Hours Reactor Was Critical	641.3	2,333.2	12,498.4
13. Reactor Reserve Shutdown Hours	4.9	41.9	190.2
14. Hours Generator On-Line	633.7	2,176.4	12,229.5
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,703,498	5,410,269	31,690,182
17. Gross Electrical Energy (MWH)	535,304	1,695,812	10,042,161
18. Net Electrical Energy (MWH)	505,742	1,593,776	9,447,222
19. Unit Service Factor	88.1	75.0	73.3
20. Unit Availability Factor	88.1	75.0	73.3
21. Unit Capacity Factor (MDC Net)	78.3	61.1	63.1
22. Unit Capacity Factor (DER Net)	77.6	60.5	62.4
23. Unit Forced Outage Rate	11.9	8.2	10.1

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): MAY 16 - MAY 26, 1980: ROUTINE MAINTENANCE.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-12	04/03/80	F	81.0	H	3	80-37	SF	VALVEX	HIGH STEAM FLOW AND LOW STEAM PRESSURE SAFETY INJECTION WITH REACTOR TRIP. WHILE ATTEMPTING TO PERFORM A 3 DEG STROKE TEST ON "A" MAIN STEAM LINE TRIP VALVE THE VALVE WENT FULLY CLOSED. RESET ALL MSIV'S TO PROPER DEGREE OF STROKE.
80-13	04/08/80	F	4.3	G	3				TURBINE TRIP CAUSED REACTOR TRIP. ELECTRICAL HYDRAULIC CONTROL SYSTEM RESERVOIR LO-LO LEVEL ALARM ACTUATION. LIMIT SWITCH WAS INADVERTANTLY OPERATED BY TURBINE OPERATOR WHILE ATTEMPTING TO DETERMINE CAUSE FOR HI LEVEL ALARM.

SUMMARY

OPERATION WAS ROUTINE DURING THE REPORT PERIOD, EXCEPT FOR AN OUTAGE FROM 04/03/80 THROUGH 04/06/80 WHEN A SAFETY INJECTION AND REACTOR TRIP OCCURRED DURING A STROKE TEST ON THE "A" MAIN STEAM LINE TRIP VALVE. A TURBINE/REACTOR TRIP OCCURRED ON 04/08/80 FROM INADVERTENT OPERATION OF THE LO-LO ALARM DURING INVESTIGATION OF THE HIGH LEVEL ALARM IN THE ELECTROHYDRAULIC CONTROL SYSTEM RESERVOIR.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

OCONEE 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	South Carolina/Oconee 30 miles W of Greenville
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2568
5.	Design Electrical Rating (MWE Net)	887
6.	Date of Initial Criticality	April 19, 1973
7.	Date of Commercial Operation	July 15, 1973
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Keowee Lake

Utility & Contractor Information

10.	Licensee	Duke Power Company
11.	Corporate Address	422 S. Church Street Charlotte, North Carolina 28242
12.	Corporate Contact	W. O. Parker, Jr.
13.	Architect/Engineer	DPC and Bechtel (Containment Bldg.)
14.	Nuclear Steam Supply System	B&W
15.	Constructor	DPC

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R.C. Lewis
18.	IE Section Chief	R. MARTIN
19.	IE Resident Inspector	F. Jape (803) 882-6927
20.	Licensing Project Manager	M. B. Fairtile
21.	Docket Number	50-269
22.	License Number/Date of Issue	DPR-38, 2/6/73
23.	Public Document Room	Oconee County Library 201 S. Spring Street Walhalla, South Carolina 29691

INSPECTION STATUS

ENFORCEMENT STATUS

AS REQUIRED BY TECHNICAL SPECIFICATION 6.4.1.E, MAINTENANCE ACTIVITIES WHICH COULD AFFECT SAFETY ARE REQUIRED TO BE ACCOMPLISHED BY ADHERENCE TO PROCEDURES AND INSTRUCTIONS. WORK REQUESTS 54292 AND 57139 WERE ISSUED WITH INSUFFICIENT INSTRUCTIONS TO INSURE THAT THE PROPER GASKET WOULD BE ISSUED TO WORKMEN FOR PRIMARY MANWAY INSTALLATIONS ON A OTSG. FURTHER MP/0/A/1130/2 WAS NOT FOLLOWED COMPLETELY WHICH RESULTED IN A PRIMARY SYSTEM BOUNDARY LEAK ON FEBRUARY 9, 1980, WHILE THE PLANT WAS HEATING UP IN PREPARATION FOR RETURN TO POWER OPERATION.
(8007 INFRACTION)

AS REQUIRED BY TECHNICAL SPECIFICATION 6.4.1.A PROCEDURES SHALL BE FOLLOWED FOR OPERATION OF SYSTEMS AND COMPONENTS INVOLVING NUCLEAR SAFETY OF THE FACILITY. ON FEBRUARY 5, 1980, THE EMERGENCY FEEDWATER TURBINE WAS PLACED INTO OPERATION WITHOUT VERIFYING BEARING OIL PRESSURE AS INSTRUCTED IN OP/1/A/1106/06 RESULTING IN DESTRUCTION OF THE TURBINE BEARINGS.
(8007 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

- UNIT OPERATING ON THREE REACTOR COOLANT PUMPS, LIMITING POWER TO 75%.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

- 72% POWER.
- LAST IE SITE INSPECTION DATE: APRIL 1-30, 1980
- INSPECTION REPORT NO: 50-269/80-17

SUMMARY

INSPECTION SUMMARY

INSPECTION FEBRUARY 1-29 (80-05): THIS ROUTINE INSPECTION INVOLVED 50 INSPECTOR-HOURS ON SITE IN THE AREAS OF PLANT OPERATIONS, OUTAGE ACTIVITIES, WITNESSING TESTING, COMPLIANCE WITH THE JANUARY 2 AND FEBRUARY 7, 1980 SHOW CAUSE ORDERS, PLANT TOURS, FOLLOWUP ON PREVIOUS INSPECTION FINDINGS, VERIFYING COMPLETION OF SELECTED FIRE PROTECTION COMMITMENTS, AND MEETING WITH LOCAL CIVIL DEFENCE PERSONNEL. OF THE EIGHT AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN SIX AREAS; TWO APPARENT ITEMS OF NONCOMPLIANCE WERE FOUND IN TWO AREAS (INFRACTION: FAILURE TO FOLLOW A PROCEDURE RESULTING IN DAMAGE TO STEAM DRIVEN EMERGENCY FEEDWATER TURBINE BEARINGS; AND INFRACTION: FAILURE TO FOLLOW PROCEDURES RESULTING IN AN INCORRECT PRIMARY GASKET TO BE INSTALLED ON A STEAM GENERATOR MANWAY).

- INSPECTION MARCH 3-31 (80-13): THIS ROUTINE INSPECTION INVOLVED 36 RESIDENT INSPECTOR-HOURS ON SITE IN THE AREAS OF PLANT OPERATIONS, OUTAGE ACTIVITIES, FOLLOWUP ON PREVIOUS INSPECTION FINDINGS, AND REVIEW OF LER'S. OF THE FOUR AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
80-04/ 03L-0	02/16/80	03/17/80	30 DAY LER	HIGH PRESSURE SERVICE WATER PUMP INOPERABLE.
80-05/ 03L-0	02/29/80	03/28/80	30 DAY LER	RPS PRESSURE CHANNEL B OUT OF CALIBRATION.
80-06/ 03L-0	03/06/80	04/04/80	30 DAY LER	NORMALLY OPEN DHR COOLER OUTLET VALVE FAILED TO OPEN DURING TEST DUE TO BAD LIMIT SWITCH. LPI TRAIN B INOP FOR 13 MIN IS OPERATING IN DEGRADED MODE PERMITTED IN LCO.

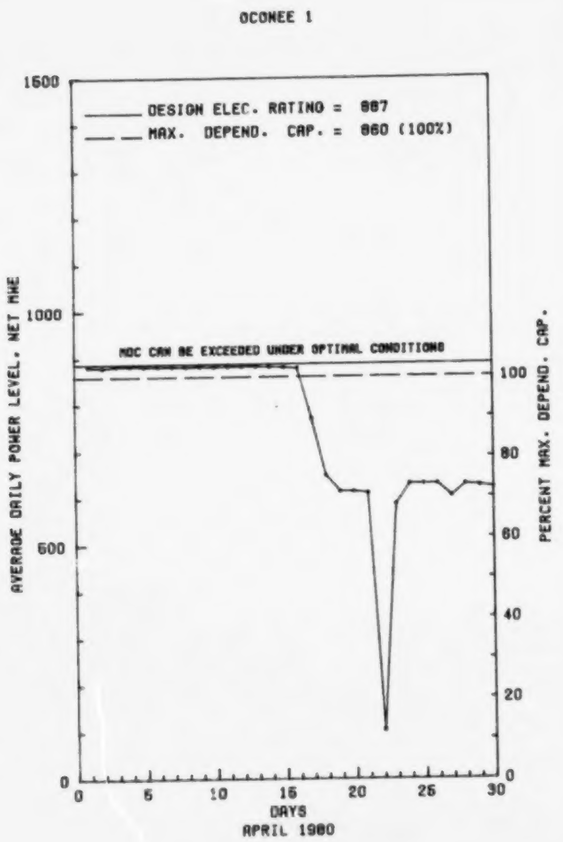
REVIEWED BY OMPA LOEB: F L Bobe **DATE:** 6/12/80

REVIEWED BY NRR DOR: M. B. Fairtile **DATE:** 6/10/80

GPO 875-667

OCONEE 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-269
2. Utility Contact: J. A. Reavis (704) 373-8552
3. Licensed Thermal Power (MWt): 2568
4. Nameplate Rating (Gross MWe): 1038 x 0.9 = 934
5. Design Electrical Rating (Net MWe): 887
6. Maximum Dependable Capacity (Gross MWe): 899
7. Maximum Dependable Capacity (Net MWe): 860

CAPACITY FACTORS CALCULATED WITH A WEIGHTED AVERAGE

8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	719.0	2,903.0	59,546.0
12. Hours Reactor Was Critical	719.0	1,577.5	42,092.2
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	706.0	1,512.8	39,459.9
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,569,628	3,437,776	92,655,895
17. Gross Electrical Energy (MWH)	563,110	1,229,980	32,145,280
18. Net Electrical Energy (MWH)	537,781	1,163,301	30,394,802
19. Unit Service Factor	98.2	52.1	66.3
20. Unit Availability Factor	98.2	52.1	66.3
21. Unit Capacity Factor (MDC Net)	87.0	46.6	58.1
22. Unit Capacity Factor (DER Net)	84.3	45.2	57.5
23. Unit Forced Outage Rate	1.8	17.7	18.0

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): ELEC MOD - 6/22/80; SAFE SHUTDOWN MOD - 9/7/80.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
5	04/17/80	F	.0	A	5		CB	MOTORX	LOW OIL LEVEL AND ERRATIC TEMPERATURES ON 1B1 RCP MOTOR BOTTOM BEARING. POWER REDUCED AND PUMP REMOVED FROM SERVICE
5A	04/22/80	F	8.3	A	1		CB	MOTORX	LOW OIL LEVEL ON 1A1 RCP MOTOR BOTTOM BEARING. UNIT WAS SHUTDOWN TO ADD OIL.
6	04/22/80	F	4.7	A	1		HA	TURBIN	HIGH TURBINE BEARING VIBRATION. UNIT WAS SHUTDOWN AND RESTARTED.
7	04/23/80	F	.0	A	5		CB	MOTORX	POWER REDUCTION CONTINUES. THE 1B1 RCP MOTOR WILL REQUIRE A LENGTHY OUTAGE FOR INSPECTION BEFORE RESTARTING.

SUMMARY

OPERATION WAS AT NEAR FULL POWER UNTIL 04/17/80 WHEN LOAD WAS REDUCED TO 72% THROUGH THE END OF THE PERIOD DUE TO LOW OIL LEVEL AND ERRATIC TEMPERATURES ON THE LOWER MOTOR BEARING. THE UNIT WAS SHUTDOWN ON 04/22/80 TO ADD OIL TO A REACTOR COOLANT PUMP MOTOR LOWER BEARING, AND AGAIN DUE TO A MANUAL TURBINE TRIP AND RESTART BECAUSE OF HIGH VIBRATION.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

OCONEE 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	South Carolina/Oconee 30 miles W of Greenville
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2568
5.	Design Electrical Rating (MWE Net)	887
6.	Date of Initial Criticality	November 11, 1973
7.	Date of Commercial Operation	September 9, 1974
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Keowee Lake

Utility & Contractor Information

10.	Licensee	Duke Power Company
11.	Corporate Address	422 S. Church Street Charlotte, North Carolina 28242
12.	Corporate Contact	W. O. Parker, Jr.
13.	Architect/Engineer	DPC/Bechtel
14.	Nuclear Steam Supply System	B&W
15.	Constructor	DPC

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R.C. Lewis
18.	IE Section Chief	R. MARTIN
19.	IE Resident Inspector	F. Jape (803) 882-6927
20.	Licensing Project Manager	M. B. Fairtile
21.	Docket Number	50-270
22.	License Number/Date of Issue	DPR-47, 10/6/73
23.	Public Document Room	Oconee County Library 201 S. Spring Street Walhalla, South Carolina 29691

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

REFUELING OUTAGE.

+ LAST IE SITE INSPECTION DATE: APRIL 1-30, 1980

+ INSPECTION REPORT NO: 50-270/80-12

SUMMARY

INSPECTION SUMMARY

INSPECTION FEBRUARY 1-29 (80-05): THIS ROUTINE INSPECTION INVOLVED 50 INSPECTOR-HOURS ON SITE IN THE AREAS OF PLANT OPERATIONS, OUTAGE ACTIVITIES, WITNESSING TESTING, COMPLIANCE WITH THE JANUARY 2 AND FEBRUARY 7, 1980 SHOW CAUSE ORDERS, PLANT TOURS, FOLLOWUP ON PREVIOUS INSPECTION FINDINGS, VERIFYING COMPLETION OF SELECTED FIRE PROTECTION COMMITMENTS, AND MEETING WITH LOCAL CIVIL DEFENSE PERSONNEL. OF THE EIGHT AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN SIX AREAS; TWO APPARENT ITEMS OF NONCOMPLIANCE WERE FOUND IN TWO AREAS (INFRACTION: FAILURE TO FOLLOW A PROCEDURE RESULTING IN DAMAGE TO STEAM DRIVEN EMERGENCY FEEDWATER TURBINE BEARINGS; AND INFRACTION: FAILURE TO FOLLOW PROCEDURES RESULTING IN AN INCORRECT PRIMARY GASKET TO BE INSTALLED ON A STEAM GENERATOR MANWAY).

+ INSPECTION MARCH 24-26 (80-07): THIS SPECIAL, ANNOUNCED INSPECTION INVOLVED 38 INSPECTOR-HOURS ON SITE IN THE AREA OF INSTALLED EQUIPMENT REVIEW WITH RESPECT TO THE IE BULLETIN 79-01B RESPONSE. OF THE AREA INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

+ INSPECTION MARCH 3-31 (80-08): THIS ROUTINE INSPECTION INVOLVED 36 RESIDENT INSPECTOR-HOURS ON SITE IN THE AREAS OF PLANT OPERATIONS, OUTAGE ACTIVITIES, FOLLOWUP ON PREVIOUS INSPECTION FINDINGS, AND REVIEW OF LER'S. OF THE FOUR AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

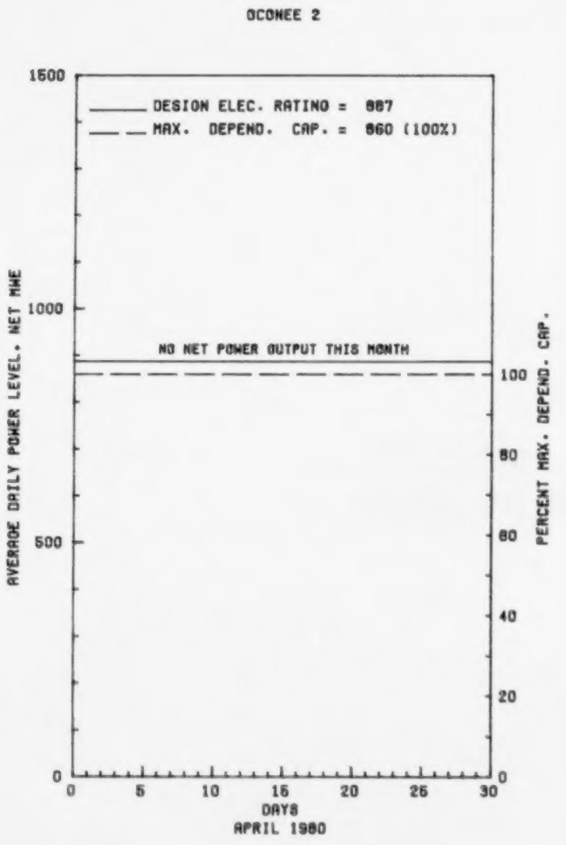
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
NONE				

REVIEWED BY OMPA LOEB: RE Bobe **DATE:** 6/12/80

REVIEWED BY NRR DOR: M.B. Fairtile **DATE:** 6/16/80

OCONEE 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-270
2. Utility Contact: J. A. Reavis (704) 373-8552
3. Licensed Thermal Power (MWt): 2568
4. Nameplate Rating (Gross MWe): 1038 x 0.9 = 934
5. Design Electrical Rating (Net MWe): 887
6. Maximum Dependable Capacity (Gross MWe): 899
7. Maximum Dependable Capacity (Net MWe): 860

CAPACITY FACTORS CALCULATED WITH A WEIGHTED AVERAGE

8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	719.0	2,903.0	49,464.0
12. Hours Reactor Was Critical	.0	1,536.3	35,132.3
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	.0	1,521.8	34,297.9
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	3,326,000	81,373,404
17. Gross Electrical Energy (MWH)	0	1,126,140	27,650,496
18. Net Electrical Energy (MWH)	-1,443	1,067,543	26,271,381
19. Unit Service Factor	.0	52.4	69.3
20. Unit Availability Factor	.0	52.4	69.3
21. Unit Capacity Factor (MDC Net)	.0	42.8	61.3
22. Unit Capacity Factor (DER Net)	.0	41.5	59.8
23. Unit Forced Outage Rate	.0	.0	19.3

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 06/18/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
3D	03/04/80	S	719.0	C	4		RC	FUELXX	SCHEDULED REFUELING. NOTE: PIPE HANGER/SUPPORT INSPECTION AND MODIFICATION CONTINUES ALSO.

SUMMARY

THE UNIT REMAINED SHUTDOWN THROUGHOUT THE REPORT PERIOD FOR REFUELING AND MAINTENANCE.

LEGEND

[1] Type:
F–Forced
S–Scheduled

[2] Reason:
A–Equipment Failure
B–Maintenance or Test
C–Refueling
D–Regulatory Restriction
E–Operator Training & License Examination
F–Administrative
G–Operational Error
H–Other

[3] Method:
1–Manual
2–Manual Scram
3–Automatic Scram
4–Continuations
5–Load Reduction
9–Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

OCONEE 3

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	South Carolina/Oconee 30 miles W of Greenville
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2568
5.	Design Electrical Rating (MWE Net)	887
6.	Date of Initial Criticality	September 5, 1974
7.	Date of Commercial Operation	December 16, 1974
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Keowee Lake

Utility & Contractor Information

10.	Licensee	Duke Power Company
11.	Corporate Address	422 S. Church Street Charlotte, North Carolina 28242
12.	Corporate Contact	W. O. Parker, Jr.
13.	Architect/Engineer	DPC/Bechtel
14.	Nuclear Steam Supply System	B&W
15.	Constructor	DPC

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R.C. Lewis
18.	IE Section Chief	R. MARTIN
19.	IE Resident Inspector	F. Jape (803) 882-6927
20.	Licensing Project Manager	M. B. Fairtile
21.	Docket Number	50-287
22.	License Number/Date of Issue	DPR-55, 7/19/74
23.	Public Document Room	Oconee County Library 201 S. Spring Street Walhalla, South Carolina 29691

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

SHUTDOWN TO INSTALL MODIFICATIONS REQUIRED BY NUREG 0578.

◆ LAST IE SITE INSPECTION DATE: APRIL 1-30, 1980

◆ INSPECTION REPORT NO: 50-287/80-11

SUMMARY

INSPECTION SUMMARY

INSPECTION FEBRUARY 1-29 (80-05): THIS ROUTINE INSPECTION INVOLVED 50 INSPECTOR-HOURS ON SITE IN THE AREAS OF PLANT OPERATIONS, OUTAGE ACTIVITIES, WITNESSING TESTING, COMPLIANCE WITH THE JANUARY 2 AND FEBRUARY 7, 1980 SHOW CAUSE ORDERS, PLANT TOURS, FOLLOWUP ON PREVIOUS INSPECTION FINDINGS, VERIFYING COMPLETION OF SELECTED FIRE PROTECTION COMMITMENTS, AND MEETING WITH LOCAL CIVIL DEFENCE PERSONNEL. OF THE EIGHT AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN SIX AREAS; TWO APPARENT ITEMS OF NONCOMPLIANCE WERE FOUND IN TWO AREAS (INFRACTION: FAILURE TO FOLLOW A PROCEDURE RESULTING IN DAMAGE TO STEAM DRIVEN EMERGENCY FEEDWATER TURBINE BEARINGS; AND INFRACTION: FAILURE TO FOLLOW PROCEDURES RESULTING IN AN INCORRECT PRIMARY GASKET TO BE INSTALLED ON A STEAM GENERATOR MANWAY).

INSPECTION MARCH 24-26 (80-07): THIS SPECIAL, ANNOUNCED INSPECTION INVOLVED 38 INSPECTOR-HOURS ON SITE IN THE AREA OF INSTALLED EQUIPMENT REVIEW WITH RESPECT TO THE IE BULLETIN 79-01B RESPONSE. OF THE AREA INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

◆ INSPECTION MARCH 3-31 (80-08): THIS ROUTINE INSPECTION INVOLVED 36 RESIDENT INSPECTOR-HOURS ON SITE IN THE AREAS OF PLANT OPERATIONS, OUTAGE ACTIVITIES, FOLLOWUP ON PREVIOUS INSPECTION FINDINGS, AND REVIEW OF LER'S. OF THE FOUR AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
80-04/ 03L-0	02/25/80	03/27/80	30 DAY LER	BREACH DISCOVERED IN CABLE PENETRATION FIRE BARRIER.
80-05/ 03L-0	03/02/80	03/31/80	30 DAY LER	INOPERABLE BORATED WATER STORAGE TANK LEVEL CHANNEL.

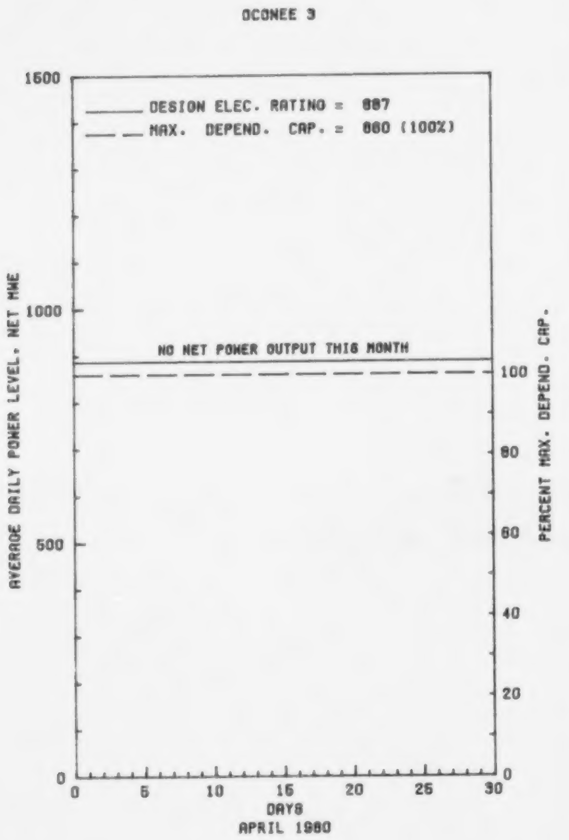
REVIEWED BY OMPA LOEB: P E Bobe **DATE:** 6/12/80

REVIEWED BY NRR DOR: M. B. Fairtile **DATE:** 6/16/80

GPO 871-867

OCONEE 3

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-287
2. Utility Contact: J. A. Reavis (704) 373-8552
3. Licensed Thermal Power (MWt): 2568
4. Nameplate Rating (Gross MWe): 1038 x 0.9 = 934
5. Design Electrical Rating (Net MWe): 887
6. Maximum Dependable Capacity (Gross MWe): 899
7. Maximum Dependable Capacity (Net MWe): 860
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

CAPACITY FACTORS CALCULATED WITH A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	719.0	2,903.3	47,111.0
12. Hours Reactor Was Critical	23.5	1,789.6	33,683.5
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	.0	1,766.1	32,828.0
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	8,052	4,300,270	78,881,875
17. Gross Electrical Energy (MWH)	0	1,550,610	27,301,874
18. Net Electrical Energy (MWH)	-7,396	1,476,556	25,973,112
19. Unit Service Factor	.0	60.8	69.7
20. Unit Availability Factor	.0	60.8	69.7
21. Unit Capacity Factor (MDC Net)	.0	59.1	63.8
22. Unit Capacity Factor (DER Net)	.0	57.3	62.2
23. Unit Forced Outage Rate	100.0	8.3	17.6

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 5/15/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
5A	03/15/80	S	568.0	D	4		ZZ	ZZZZZZ	NRC REQUIRED MODIFICATIONS NUREG 578 AND OTHER NSM'S CONTINUE.
5B	04/24/80	F	89.3	A	9		CA	PIPEXX	PRESSURIZER RELIEF VALVE (3RC-67) FLANGE LEAK DELAYED UNIT STARTUP.
5C	04/28/80	F	52.4	A	9		CF	VALVOP	CONTROLLER ON (3 LPSW-404) DECAY HEAT REMOVAL COOLER OUTLET VALVE CAUSED HOLD ON STARTUP.
5D	04/30/80	F	9.3	G	9		HA	GENERA	FAILURE TO REMOVE GROUND STRAPS ON GENERATOR PHASES BEFORE CLOSING THE EXCITER FIELD BREAKER NECESSITATED INSPECTION OF BUS BEFORE PUTTING UNIT IN SERVICE.

SUMMARY

THE UNIT REMAINED SHUTDOWN UNTIL 04/28/80 TO MAKE MODIFICATIONS REQUIRED BY NUREG-0578. STARTUP WAS DELAYED ON 04/24/80 WHEN A FLANGE LEAK REQUIRED COOLDOWN FOR REPAIR; ON 04/28/80, WHEN A VALVE OPERATOR ON THE HEAT DECAY COOLER OUTLET CAUSED A HOLD; AND ON 04/30/80 WHEN FAILURE TO REMOVE GENERATOR BUS GROUND STRAPS BEFORE CLOSING THE GENERATOR FIELD BREAKER REQUIRED INVESTIGATION OF POSSIBLE BUS DAMAGE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

OYSTER CREEK

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	New Jersey/Ocean 9 miles S of Toms River
2.	Electric Reliability Council	Mid-Atlantic Area Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	1930
5.	Design Electrical Rating (MWE Net)	650
6.	Date of Initial Criticality	May 3, 1969
7.	Date of Commercial Operation	December 1969
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Bernegat Bay

Utility & Contractor Information

10.	Licensee	Jersey Central Power & Light Co.
11.	Corporate Address	Madison Avenue At Punch Bowl Road Morristown, New Jersey 07960
12.	Corporate Contact	I. Finfrock, Vice President Power Generation
13.	Architect/Engineer	Burns & Roe
14.	Nuclear Steam Supply System	GE
15.	Constructor	J. A. Jones

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	IE Section Chief	R. R. KEIMIG
19.	IE Resident Inspector	L. BRIGGS (X5306) (609) 693-0702 NRC RESIDENT OFFICE
20.	Licensing Project Manager	W. A. Paulson
21.	Docket Number	50-219
22.	License Number/Date of Issue	DPR-16, 4/9/69
23.	Public Document Room	Ocean County Library 15 Hooper Avenue Toms River, New Jersey 08753

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS:

+ Shutdown for refueling, with return to operation scheduled June 5, 1980.

+ LAST IE SITE INSPECTION DATE: 4/18/80

+ INSPECTION REPORT NO: 50-219/80-14

SUMMARY

INSPECTION SUMMARY

+ 50-219/80-09 - Mar 11-14: Routine, onsite, unannounced inspection by two region-based inspectors (40 hours) of three inspection areas: inservice inspection data review meeting; independent inspection effort; and followup on regional request. A plant tour was also conducted. Noncompliances: None.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-012/03L	02/28/80	03/27/80	Weekly surveillance was missed (once) for the diesel anu station battery

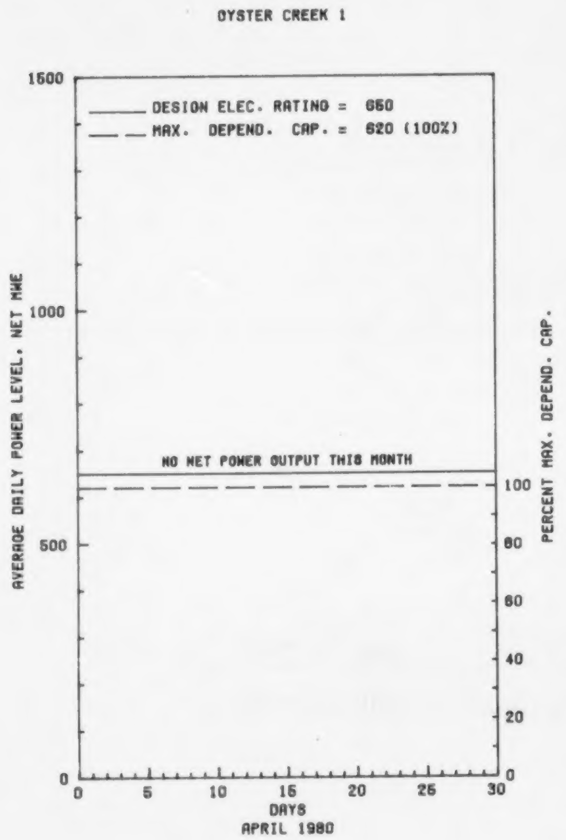
REVIEWED BY OMPA LOEB: ____________ **DATE:** ____________

REVIEWED BY NRR DOR: Walter A. Paulson **DATE:** 6/16/80

GPO 871-887

OYSTER CREEK

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-219
2. Utility Contact: C. M. McClain (201) 455-8748
3. Licensed Thermal Power (MWt): 1930
4. Nameplate Rating (Gross MWe): 687.5 x 0.8 = 550
5. Design Electrical Rating (Net MWe): 650
6. Maximum Dependable Capacity (Gross MWe): 650
7. Maximum Dependable Capacity (Net MWe): 620
8. If Changes Occur In Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

ITEM 21 CALCULATED BY USING A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	719.0	2,903.0	90,767.0
12. Hours Reactor Was Critical	.0	101.5	68,740.8
13. Reactor Reserve Shutdown Hours	.0	.0	468.2
14. Hours Generator On-Line	.0	98.4	67,403.9
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	157,611	116,036,769
17. Gross Electrical Energy (MWH)	0	53,670	38,935,985
18. Net Electrical Energy (MWH)	0	48,612	37,529,625
19. Unit Service Factor	.0	3.4	74.3
20. Unit Availability Factor	.0	3.4	74.3
21. Unit Capacity Factor (MDC Net)	.0	2.7	68.3
22. Unit Capacity Factor (DER Net)	.0	2.6	63.6
23. Unit Forced Outage Rate	.0	.0	6.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 6/5/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	01/05/80	S	719.0	C	4		ZZ	ZZZZZZ	REFUELING/MAINTENANCE OUTAGE.

SUMMARY

THE UNIT REMAINED SHUTDOWN THROUGHOUT THE REPORT PERIOD FOR REFUELING AND MAINTENANCE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

PALISADES

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Michigan/Van Buren 5 miles S of South Haven
2.	Electric Reliability Council	East Central Area Reliability Coordination Agreement
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2530
5.	Design Electrical Rating (MWE Net)	805
6.	Date of Initial Criticality	May 24, 1971
7.	Date of Commercial Operation	December 31, 1971
8.	Condenser Cooling Method	Forced Draft Cooling Towers
9.	Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10.	Licensee	Consumers Power Company
11.	Corporate Address	212 W. Michigan Avenue Jackson, Michigan 49201
12.	Corporate Contact	R. B. DeWitt
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Combustion Engineering
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	D.C. Boyd
19.	IE Resident Inspector	B. Jorgenson (616) 764-8971
20.	Licensing Project Manager	T. Wambach
21.	Docket Number	50-255
22.	License Number/Date of Issue	DPR-20, 3/24/71
23.	Public Document Room	Kalamazoo Public Library 315 South Rose Street Reference Department Kalamazoo, Michigan 49006

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

Reactor shutdown on September 8, 1979, to begin a 10-week scheduled refueling amd maintenance outage. Start-up tentatively scheduled for mid-May 1980.

+LAST IE SITE INSPECTION DATE: February 1980

+ INSPECTION REPORT NO: 80-03

SUMMARY

INSPECTION SUMMARY

+ Inspection during February 1980, (80-03): Routine resident inspection program activities during February 1980, including: inspection during long-term shutdown; reportable occurrences; and actions on IE Bulletins and NUREG-0578 items. The inspection involved 78 inspector hours onsite by two NRC inspectors. No items of noncompliance or deviations were identified in the areas inspected.

REPORTS RECEIVED FROM LICENSEE

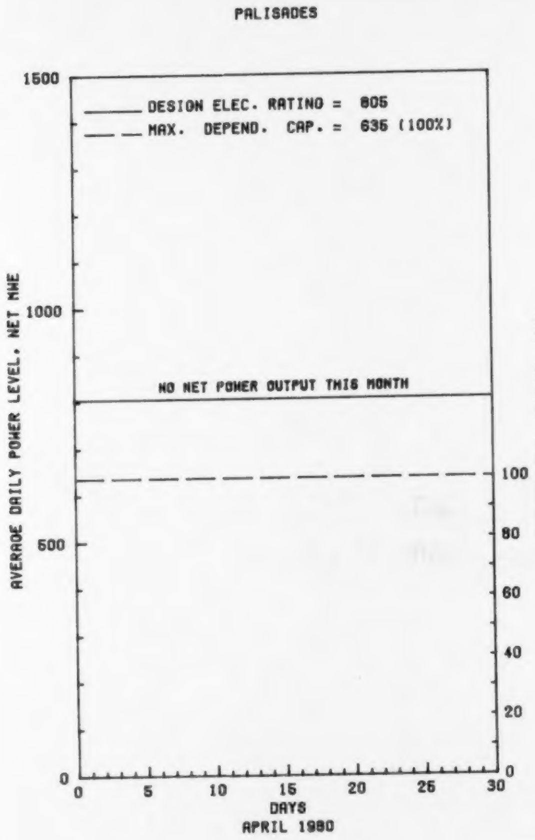
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-02/ 03L-0	10/23/79	04/03/80	During routine testing, snubber No. 5-6, on main steam line, failed to meet the test acceptance criteria.
80-03/ 01T-0	03/04/80	04/03/80	During evaluation of SEP Topic RE HELB, it was determined that incorrect assumptions for initiztion of containment spray as well as inaccuracies in SG mass-energy inventories resulted in predicted peak containment pressures being lower than actual.
80-04/ 01T-0	03/27/80 he	04/10/80	During review of TS Surveillance Procedures, it was discovered that procedure MO-19 required isolation of both containment spray headers during performance of the test.
80-05/ 01T0	03/27/80	04/10/80	During review of TS Surveillance Procedures, it was discovered that two procedures (QO-2 and MO-23) require stroking the SIRW Tank oulet valves.
80-06/ 03L-0	11/10/79	04/21/80	During routine surveillance of containment purge valves, it was found that the redundant T-ring air supplies for the 12-inch and 48-inch purge valves were leaking in excess of criteria.
80-07/ 01T-0	04/07/80	04/21/80	During maintenance on the control room ventilation system, two defects which would have permitted large inleakage of contamination into the control room under accident conditions were discovered.

REVIEWED BY OMPA LOEB: F.E. Bobe **DATE:** 6/12/80

REVIEWED BY NRR DOR: T.V. Wambach **DATE:** 6/16/80

PALISADES

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-255
2. Utility Contact: P. J. Bodtke (616) 764-8913
3. Licensed Thermal Power (MWt): 2530
4. Nameplate Rating (Gross MWe): 955 x 0.85 = 812
5. Design Electrical Rating (Net MWe): 805
6. Maximum Dependable Capacity (Gross MWe): 675
7. Maximum Dependable Capacity (Net MWe): 635
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	719.0	2,903.0	73,334.0
12. Hours Reactor Was Critical	.0	.0	39,564.8
13. Reactor Reserve Shutdown Hours	.0	.0	0
14. Hours Generator On-Line	.0	.0	37,426.5
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	0	71,511,480
17. Gross Electrical Energy (MWH)	0	0	22,032,230
18. Net Electrical Energy (MWH)	0	0	20,670,775
19. Unit Service Factor	.0	.0	51.0
20. Unit Availability Factor	.0	.0	51.0
21. Unit Capacity Factor (MDC Net)	.0	.0	44.4
22. Unit Capacity Factor (DER Net)	.0	.0	35.0
23. Unit Forced Outage Rate	.0	.0	36.2

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 05/15/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
13	09/08/79	S	719.0	D	4				TMI MODIFICATIONS AND ANCHOR BOLT INSPECTIONS/REPAIRS ACCORDING TO IE BULLETIN 79-02.

SUMMARY

THE UNIT REMAINED SHUTDOWN FOR SEISMIC PIPE HANGER/ANCHOR BOLT INSPECTIONS AND REPAIR, AND FOR TMI-RELATED MODIFICATIONS.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

PEACH BOTTOM 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1. Location (state/county) (nearest population center): Pennsylvania/York
19 miles S of Lancaster
2. Electric Reliability Council: Mid-Atlantic Area Council
3. Type of Reactor: BWR
4. Power Level (MWT): 3293
5. Design Electrical Rating (MWE Net): 1065
6. Date of Initial Criticality: Septemb~~: 16, 1973
7. Date of Commercial Operation: July 5, 1974
8. Condenser Cooling Method: Once Through with Cooling Towers
9. Condenser Cooling Water Source: Conowingo Pond/Susquehanna River

Utility & Contractor Information

10. Licensee: Philadelphia Electric Company
11. Corporate Address: 2301 Market Street, Philadelphia, Pennsylvania 19101
12. Corporate Contact: S. L. Daltroff, Vice President, Electric Production
13. Architect/Engineer: Bechtel
14. Nuclear Steam Supply System: GE
15. Constructor: Bechtel

Regulatory Information

16. IE Region responsible: I
17. IE Branch Chief: E. Brunner
18. IE Section Chief: E. McCabe
19. IE Resident Inspector: C. Cowgill 717-456-7614
20. Licensing Project Manager: D. Verrelli
21. Docket Number: 50-277
22. License Number/Date of Issue: DPR-44, 8/8/73
23. Public Document Room:

Government Publications Section
State Library of Pennsylvania
Education Building
Commonwealth and Walnut Street
Harrisburg, Pennsylvania 17126

P-4

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO THE ABOVE, ON 1/3/80, WITH UNIT 2 IN POWER OPERATION & THE DRYWELL BEING DEINERTED, 2 DRYWELL REQUIREMENTS HAD BEEN IMPOSED TO RESTRICT OPENING OF CONTAINMENT VENTILATION SYSTEM VALVES TO AN OPENING WHEREBY CLOSURE AGAINST POST LOCA CONDITIONS IS ASSURED. FURTHER, THE ENGINEERING REVIEW & IDENTIFICATION OF THIS DEFICIENCY HAD BEEN COMPLETED ON 3/2/79 & CORRECTIVE ACTION DESCRIBED IN PECO LTR TO RI DTD 3/29/79 STATED THAT OPENINGS OF THESE VALVES ARE RESTRICTED, THAT THE SUPPLY PRESSURE TO THE VALVES WAS ADJUSTED SO THAT THE VALVES WOULD ONLY OPEN TO 30 DEGREES & THAT THESE OPERATIONS WERE ADDRESSED IN A SYSTEM PROCEDURE.
(8004 INFRACTION)

ON 2/22/80 A ROUND SHEET DEVELOPED TO PROVIDE ADMINISTRATIVE CONTROL OVER CONTAINMENT VENTILATION VALVE SEISMIC CAPABILITY DESIGNED TO ASSURE CONTAINMENT INTEGRITY WAS BEING USED WITHOUT INCORPORATION INTO SYSTEM PROCEDURES & WITHOUT PORC REVIEW & APPROVAL.
(8004 INFRACTION)

TS 6.8.1 STATES IN PART, "WRITTEN PROCEDURES & ADMINISTRATIVE POLICIES SHALL BE ESTABLISHED, IMPLEMENTED & MAINTAINED..." PROCEDURE A-14, "PLANT MODIFICIATION, REV 6, DTD 8/8/78 & SUBSEQUENT REVISIONS, STATES IN PART, "THE MODIFICATION COORDINATION SHALL...DOCUMENT THE NEED FOR PROCEDURE...REVISIONS ON THE PCMS." CONTRARY TO THE ABOVE, THE NEED FOR REQUIRED PROCEDURAL REVISIONS WAS NOT DOCUMENTED ON THE PMCS FOR MOD 510 (79-29), INITIATED 3/16/79. PROCEDURE A-14, "PLANT MODIFICATION, REV 6, DTD 8/8/78 & SUBSEQUENT REVISIONS, STATES IN PART, "AN INDEPENDENT DESIGN REVIEW SHALL BE...DOCUMENTED BY SIGNING...THE PMCS."
(8004 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

MANAGERIAL ITEMS:

- Mr. V. S. Boyer was appointed Sr. Vice President, Nuclear Power. His role is an overall coordination of the company's nuclear activities. Mr. John H. Austin, the Executive Vice President, was designated Chief Executive Officer. Mr. John S. Kemper was promoted from Manager, Engineering and Research, to Vice President, Engineering and Research.
- On April 24, the NRC conducted a technical meeting with the licensee and a public meeting to discuss upgraded emergency planning criteria.

PLANT STATUS:

- The plant continued a scheduled refueling outage. All fuel was off-loaded to the fuel pool. Reactor vessel in-service inspection and feedwater sparger replacement were begun. The suppression chamber (torus) was dewatered for torus structural modifications.

- LAST IE SITE INSPECTION DATE: 4/30/80
- INSPECTION REPORT NO: 50-277/80-07

SUMMARY

INSPECTION SUMMARY

- 50-277/79-09 - Mar 19-30: Special, announced inspection of the licensee's management controls over licensed activities. The inspection involved 168 inspector-hours onsite and 136 inspector-hours in the corporate offices by four NRC inspectors. Noncompliances: Three (failure to provide an adequate retraining program for non-licensed personnel; failure to perform quality assurance audits; and failure to follow procedures). Deviations: One (unqualified receipt inspectors).
- 50-277/80-06 - Mar 17-21: Routine, unannounced inspection (17 hours) by a region-based inspector of administrative controls for surveillance procedures; surveillance testing; inspector witnessing of surveillance testing; technician qualification; and facility tour observations. Noncompliances: One (failure to properly document two surveillance tests and corrective maintenance; and failure to record Technical Specification required data).

REPORTS RECEIVED FROM LICENSEE

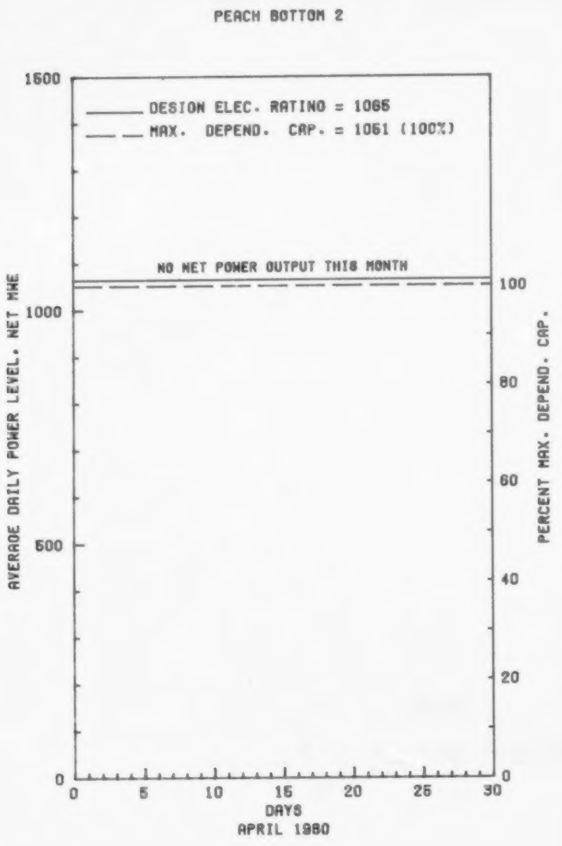
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-006/03L	03/05/80	04/03/80	1 of 2 offsite power sources due to a loss of transformer fans causing a high temperature trip. Unit 2 recovered w/o problems and Unit 3 scrammed during recovery

REVIEWED BY OMPA LOEB: P. E. Bob **DATE:** 6/12/80

REVIEWED BY NRR DOR: D M Verrelli **DATE:** 6/13/80

PEACH BOTTOM 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-277
2. Utility Contact: W. M. Alden (215) 841-5022
3. Licensed Thermal Power (MWt): 3293
4. Nameplate Rating (Gross MWe): 1280 x 0.9 = 1152
5. Design Electrical Rating (Net MWe): 1065
6. Maximum Dependable Capacity (Gross MWe): 1098
7. Maximum Dependable Capacity (Net MWe): 1051
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	719.0	2,903.0	51,047.0
12. Hours Reactor Was Critical	.0	1,441.0	39,196.0
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	.0	1,390.2	38,373.0
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	3,831,770	110,597,414
17. Gross Electrical Energy (MWH)	0	1,255,980	36,314,210
18. Net Electrical Energy (MWH)	-7,517	1,195,873	34,811,658
19. Unit Service Factor	.0	47.9	75.2
20. Unit Availability Factor	.0	47.9	75.2
21. Unit Capacity Factor (MDC Net)	.0	39.2	64.9
22. Unit Capacity Factor (DER Net)	.0	38.7	64.0
23. Unit Forced Outage Rate	.0	.9	6.1

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 07/13/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
9	03/21/80	S	719.0	C	4		RC	FUELXX	CONTINUATION OF REFUEL OUTAGE.

SUMMARY

THE UNIT REMAINED SHUTDOWN THROUGHOUT THE REPORT PERIOD FOR REFUELING.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

PEACH BOTTOM 3

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1. Location (state/county) (nearest population center)	Pennsylvania/York 19 miles S of Lancaster
2. Electric Reliability Council	Mid-Atlantic Area Council
3. Type of Reactor	BWR
4. Power Level (MWT)	3293
5. Design Electrical Rating (MWE Net)	1065
6. Date of Initial Criticality	August 7, 1974
7. Date of Commercial Operation	December 23, 1974
8. Condenser Cooling Method	Once Through with Cooling Towers
9. Condenser Cooling Water Source	Conowingo Pond/Susquehanna River

Utility & Contractor Information

10. Licensee	Philadelphia Electric Company
11. Corporate Address	2301 Market Street Philadelphia, Pennsylvania 19101
12. Corporate Contact	S. L. Daltroff, Vice President Electric Production
13. Architect/Engineer	Bechtel
14. Nuclear Steam Supply System	GE
15. Constructor	Bechtel

Regulatory Information

16. IE Region responsible	I
17. IE Branch Chief	E. Brunner
18. IE Section Chief	E. McCabe
19. IE Resident Inspector	C. Cowgill (717-456-7614)
20. Licensing Project Manager	D. Verrelli
21. Docket Number	50-278
22. License Number/Date of Issue	DPR-56, 7/2/74
23. Public Document Room	Government Publications Section State Library of Pennsylvania Education Building Commonwealth and Walnut Street Harrisburg, Pennsylvania 17126

INSPECTION STATUS

ENFORCEMENT STATUS

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

- Repair of a cracked weld in the suction piping of residual heat removal pump No. 3D was accomplished between 4/14 and 4/20; the crack had initiated in a weld attaching a 1-inch drain line and had propagated a short distance (less than 2 inches) into the pipe.

MANAGERIAL ITEMS:

- Mr. V. S. Boyer was appointed Sr. Vice President, Nuclear Power. His role is an overall coordination of the company's nuclear activities. Mr. John H. Austin, the Executive Vice President, was designated the Chief Executive Officer. Mr. John S. Kemper was promoted from Manager, Engineering and Research, to Vice President, Engineering and Research.
- The NRC conducted a technical meeting with the licensee and a public meeting on April 24 to discuss upgraded emergency planning criteria.

PLANT STATUS:

- The reactor operated at a nominal 100 percent power during the month. Representative offgas and combined Unit 2 and Unit 3 stack gas release rates were 3420 microcuries per second and 187 microcuries per second, respectively.

- LAST IE SITE INSPECTION DATE: 4/30/80
- INSPECTION REPORT NO: 50-278/80-07

SUMMARY

INSPECTION SUMMARY

- 50-278/79-10 - Mar 19-30: Special, announced inspection of the licensee's management controls over licensed activities. The inspection involved 168 inspector-hours onsite and 136 inspector-hours in the corporate offices by four NRC:IE Program Appraisal Branch inspectors. Noncompliances: Three (failure to provide an adequate retraining program for non-licensed personnel, failure to perform quality assurance audits, and failure to follow procedures). Deviations: One (unqualified receipt inspectors).
- 50-278/80-06 - Mar 17-21: Routine, unannounced inspection (18 hours) by a region-based inspector of administrative controls for surveillance procedures; surveillance testing; inspector witnessing of surveillance testing; technician qualification; and facility tour observations. Noncompliances: One (failure to properly document two surveillance tests and corrective maintenance).

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-003/03L	03/03/80	04/02/80	During surveillance testing 4 LPCI "A" LPCI injection valve would not open and 3A RHR pump would not start. LPCI loop B and both core spray tested operable. LPCI loop A valve repaired.

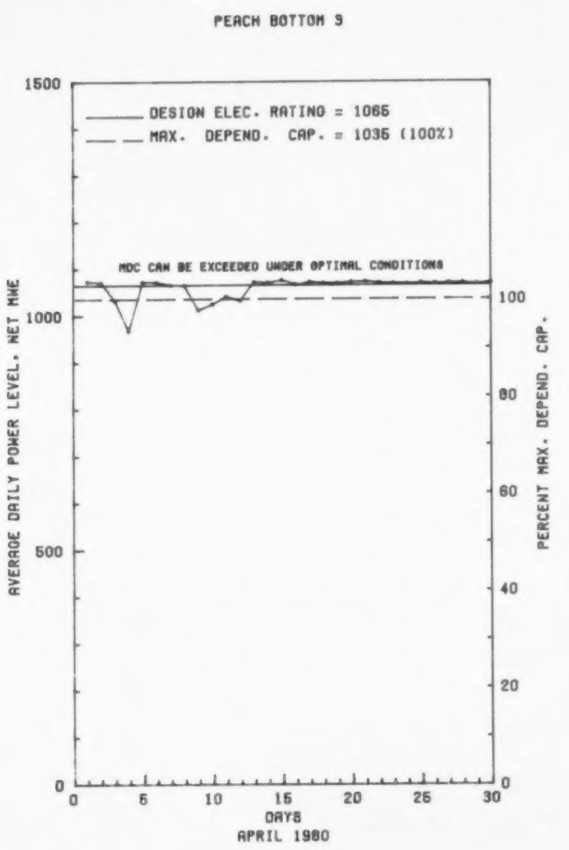
REVIEWED BY OMPA LOEB: ______ **DATE:** 6/12/80

REVIEWED BY NRR DOR: ______ **DATE:** 6/13/80

GPO 873-887

PEACH BOTTOM 3

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-278
2. Utility Contact: W. M. Alden (215) 841-5022
3. Licensed Thermal Power (MWt): 3293
4. Nameplate Rating (Gross MWe): 1280 x 0.9 = 1152
5. Design Electrical Rating (Net MWe): 1065
6. Maximum Dependable Capacity (Gross MWe): 1098
7. Maximum Dependable Capacity (Net MWe): 1035
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	719.0	2,903.0	46,943.0
12. Hours Reactor Was Critical	719.0	2,410.0	37,256.0
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	719.0	2,303.5	36,235.3
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	2,339,743	7,333,210	101,552,364
17. Gross Electrical Energy (MWH)	789,100	2,470,150	33,056,010
18. Net Electrical Energy (MWH)	761,515	2,385,028	31,729,828
19. Unit Service Factor	100.0	79.3	77.2
20. Unit Availability Factor	100.0	79.3	77.2
21. Unit Capacity Factor (MDC Net)	102.3	79.4	65.3
22. Unit Capacity Factor (DER Net)	99.4	77.1	63.5
23. Unit Forced Outage Rate	.0	6.9	6.9

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
8	04/03/80	S	.0	B	5		HA	VALVEX	LOAD DROP TO REPAIR TUBE LEAK IN 2B WATER BOX AND #3 TURBINE CONTROL VALVE.
9	04/08/80	F	.0	A	5		CH	PUMPXX	LOAD DROP FOR REACTOR FEED PUMP TRIP ON HIGH VIBRATIONS.

SUMMARY

OPERATION WAS AT NEAR FULL POWER DURING THE REPORT PERIOD, EXCEPT FOR LOAD REDUCTIONS ON 04/03/80 TO REPLACE A LIMIT SWITCH ON ONE OF THE LUBRICANT COUPLINGS ON FEED PUMPS. ON 04/08/80, A REACTOR FEED PUMP TURBINE TRIPPED ON HIGH VIBRATION.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

PILGRIM

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Massachusetts/Plymouth 4 miles SE of Plymouth
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	BWR
4.	Licenses Power Level (MWT)	1998
5.	Design Electrical Rating (MWE Net)	655
6.	Date of Initial Criticality	June 16, 1972
7.	Date of Commercial Operation	December 1972
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Cape Cod Bay

Utility & Contractor Information

10.	Licensee	Boston Edison Company
11.	Corporate Address	800 Boylston Street Boston, Massachusetts 02199
12.	Corporate Contact	A. Corey, Vice President
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	GE
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	IE Section Chief	T. MARTIN
19.	IE Resident Inspector	J. JOHNSON
20.	Licensing Project Manager	J. Hannon
21.	Docket Number	50-293
22.	License Number/Date of Issue	DPR-35, 6/8/72
23.	Public Document Room	Plymouth Public Library North Street Plymouth, Massachusetts 02360

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

- Replaced three Jet Pump hold down bars (Assemblies #1, 3, 15).
- MANAGERIAL ITEMS:
- Possible strike by represented employees on or about 5/15/80.

PLANT STATUS:

- Shutdown for refueling outage; refueling complete - startup scheduled for 5/7/80 pending completion of STLL, IEB 79-14, and ILRT.
- LAST IE SITE INSPECTION DATE: 4/22/80
- INSPECTION REPORT NO: 50-293/80-18

SUMMARY

INSPECTION SUMMARY

- 50-293/80-08 - Feb 27-29: Special inspection by one region-based inspector of the Security Program, previous items of noncompliance and IEB 79-16. One item of noncompliance was identified (failure to follow response procedures).
- 50-293/80-10 - Mar 5-7: Routine, unannounced inspection by two region-based inspectors of SNM control and accounting. No items of noncompliance were identified.
- 50-293/80-12 - Mar 5-7, 19-20: Investigation by five region-based inspectors of allegations involving shipments of spent fuel to a burial site. No items of noncompliance were identified.
- 50-293/80-15 - Feb 27-29: Unannounced inspection by one Region II inspector of activities involving a shipment of radioactive waste to a burial site. No items of noncompliance were identified.
- 50-293/80-17 - Mar 13-14: Unannounced inspection by one Region II inspector of activities involving a shipment of radioactive waste to a burial site. No items of noncompliance were identified.

REPORTS RECEIVED FROM LICENSEE

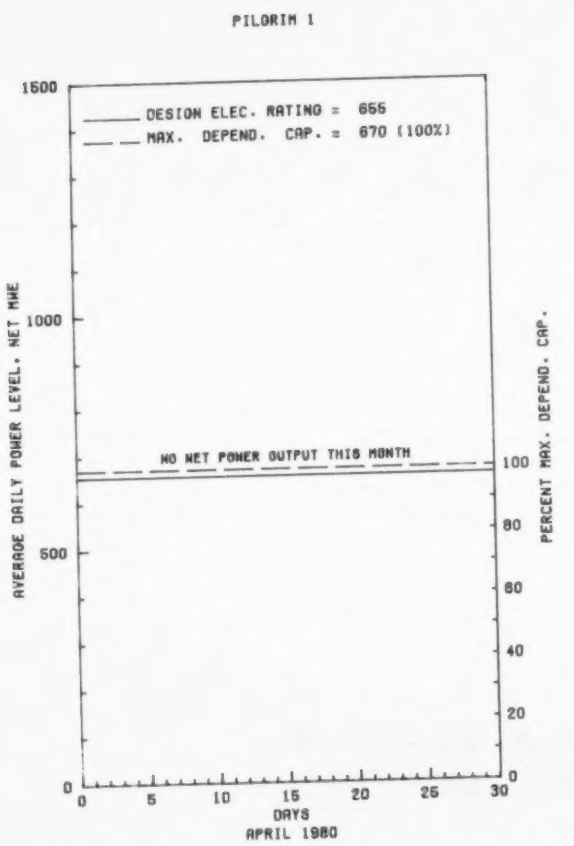
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-010/ 03L	03/10/80	04/09/80	Hydrant hose rendered inoperable as a result of truck impingement
80-011/ 01T	03/26/80	04/09/80	UT examination revealed three jet pump hold down beams to have discontinuities. Beams were replaced
80-012/ 03L	03/12/80	04/11/80	Setpoint drift low pressure permissive for opening LPCI injection valves
80-013/ 03L	03/13/80	04/11/80	Setpoint drift condenser low vacuum switch in RPS

REVIEWED BY OMPA LOEB: Sobe DATE: 6/12/80

REVIEWED BY NRR DOR: DATE: 6/13/80

PILGRIM

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-293
2. Utility Contact: G. G. Whitney (617) 746-7900
3. Licensed Thermal Power (MWt): 1998
4. Nameplate Rating (Gross MWe): 780 x 0.87 = 678
5. Design Electrical Rating (Net MWe): 655
6. Maximum Dependable Capacity (Gross MWe): 690
7. Maximum Dependable Capacity (Net MWe): 670
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	719.0	2,903.0	64,799.0
12. Hours Reactor Was Critical	.0	56.1	45,077.0
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	.0	96.1	43,648.6
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	181,152	73,272,264
17. Gross Electrical Energy (MWH)	0	61,340	24,226,174
18. Net Electrical Energy (MWH)	0	58,860	23,264, 33
19. Unit Service Factor	.0	3.3	67.4
20. Unit Availability Factor	.0	3.3	67.4
21. Unit Capacity Factor (MDC Net)	.0	3.0	53.6
22. Unit Capacity Factor (DER Net)	.0	3.1	54.8
23. Unit Forced Outage Rate	.0	.0	10.6

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 05/15/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
1	01/05/80	S	719.0	C	4	.	RC	FUELXX	REFUELING CONTINUES.

SUMMARY

THE UNIT REMAINED SHUTDOWN THROUGHOUT THE REPORT PERIOD FOR REFUELING.

LEGEND

[1] Type:
F–Forced
S–Scheduled

[2] Reason:
A–Equipment Failure
B–Maintenance or Test
C–Refueling
D–Regulatory Restriction
E–Operator Training & License Examination
F–Administrative
G–Operational Error
H–Other

[3] Method:
1–Manual
2–Manual Scram
3–Automatic Scram
4–Continuations
5–Load Reduction
9–Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

POINT BEACH 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Wisconsin/Manitowoc 15 miles N of Manitowoc
2.	Electric Reliability Council	Mid-America Interpool Network
3.	Type of Reactor	PWR
4.	Power Level (MWT)	1518
5.	Design Electrical Rating (MWE Net)	497
6.	Date of Initial Criticality	November 2, 1970
7.	Date of Commercial Operation	December 21, 1970
8.	Condenser Cooling Method	Straight Through
9.	Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10.	Licensee	Wisconsin Electric Power Co.
11.	Corporate Address	231 West Michigan Milwaukee, Wisconsin
12.	Corporate Contact	S. Burstein
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	R. F. Warnick
19.	IE Resident Inspector	J. D. SMITH
20.	Licensing Project Manager	C. M. Trammell
21.	Docket Number	50-266
22.	License Number/Date of Issue	DPR-24 issued 10/05/70
23.	Public Document Room	Document Department, Library University of Wisconsin-Stevens Point Stevens Point, Wisconsin 54481 Manitowoc Public Library Manitowoc, Wisconsin (Temporary LPDR for spent fuel pool capacity increase amendment only

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

- The unit was placed online on April 6, 1980 following an outage in March for Inservice Inspection of the Steam Generator Tubes.

- LAST IE SITE INSPECTION DATE: March 6 and 20, 1980

- INSPECTION REPORT NO: 80-05

SUMMARY

INSPECTION SUMMARY

- Inspection on January 8-11, (80-01): Routine, unannounced inspection of incore/excore calibration; isothermal temperature coefficient measurement; target axial flux difference calculation; shutdown margin determination; reactivity anomaly determination. The inspection involved 19 inspector hours onsite by one NRC inspector. No items of noncompliance or deviations were identified.

- Inspection on February 25-26, (80-03): Licensee actions relative to IE Bulletin No. 79-14, including general discussion on NRC requirements, work procedure review, and review of analysis and calculations. The inspection involved 16 inspector hours onsite by one NRC inspector. No items of non-compliance or deviations were identified.

- Inspection on February 19-20, (80-04): Routine, unannounced inspection to review plant operations, non-routine event follow-up and independent inspection. The inspection involved 15 inspector hours onsite by one NRC inspector. Of the three areas inspected, no items of noncompliance were identified in two areas. One item of noncompliance was identified in one area (minimum number of Auxiliary Feedwater Pumps not capable of automatic operation during two unit operation).

- Inspection on March 6 and 20, (80-05): QA/QC programs, implementing procedures, work activities, and records relative to the steam generators tube inspections. The inspection involved six inspector hours onsite by one NRC inspector. No items of noncompliance or deviations were identified.

REPORTS RECEIVED FROM LICENSEE

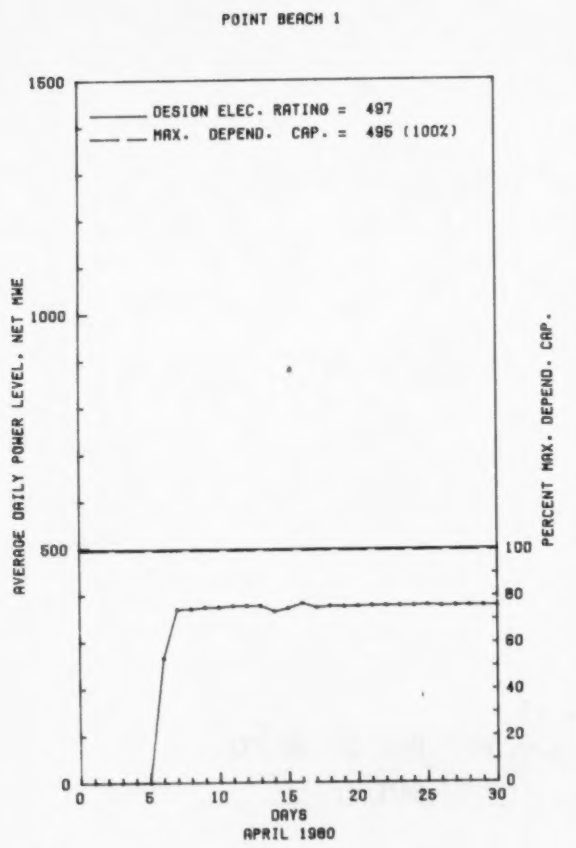
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-03/ 01T-0	03/18/80	04/16/80	Containment spray relay would not latch.

REVIEWED BY OMPA LOEB: T.C.Cintula **DATE:** 6·12·80

REVIEWED BY NRR DOR: CM Trammell **DATE:** 6/13/80

POINT BEACH 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-266
2. Utility Contact: C.W. FAY (414) 277-2811
3. Licensed Thermal Power (MWt): 1518
4. Nameplate Rating (Gross MWe): 582 x 0.9 = 524
5. Design Electrical Rating (Net MWe): 497
6. Maximum Dependable Capacity (Gross MWe): 519
7. Maximum Dependable Capacity (Net MWe): 495
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): 390
10. Reasons for Restrictions, If Any:
INHIBIT CORROSION OF STEAM GENERATOR TUBES

ITEM 21 CALCULATED BY USING A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	719.0	2,903.0	83,135.0
12. Hours Reactor Was Critical	609.8	2,154.5	68,671.0
13. Reactor Reserve Shutdown Hours	1.2	18.8	601.0
14. Hours Generator On-Line	595.8	2,011.5	66,405.5
15. Unit Reserve Shutdown Hours	4.9	151.5	753.9
16. Gross Thermal Energy (MWH)	716,495	2,413,109	93,165,904
17. Gross Electrical Energy (MWH)	233,470	787,540	31,421,290
18. Net Electrical Energy (MWH)	220,962	743,577	29,933,755
19. Unit Service Factor	82.9	69.3	79.9
20. Unit Availability Factor	83.5	74.5	80.8
*21. Unit Capacity Factor (MDC Net)	62.1	51.7	74.0
22. Unit Capacity Factor (DER Net)	61.8	51.5	72.4
23. Unit Forced Outage Rate	.0	.0	3.6

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
07/03/80 - STEAM GEN LEAK TESTING & EDDY CURRENT INSPECTION. (10 Days)
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
1	02/28/80	S	123.2	D	4	80-002	CB	HTEXCH	AS REQUIRED BY THE CONFIRMATORY ORDER OF 11/30/79, UNIT WAS TAKEN OFF LINE ON 02/28/80 FOR 60 EFFECTIVE FULL POWER DAY STEAM GENERATOR TESTING. WORK DID NOT START ON UNIT 1 UNTIL 03/10/80 BECAUSE OF A FORCED UNIT 2 OUTAGE. AN 800 PSI SECONDARY-TO-PRIMARY LEAK TEST OF BOTH GENERATORS WAS PERFORMED ON 03/11/80 AND AN EDDY CURRENT INSPECTION WAS CONDUCTED.

SUMMARY

OPERATION WAS ROUTINE FOLLOWING RETURN TO POWER ON 04/06/80 FROM THE EFFECTIVE FULL POWER 60-DAY STEAM GENERATOR TESTING AND INSPECTION OUTAGE. STEAM GENERATOR WORK INCLUDED EDDY CURRENT TESTING, EXPLOSIVE TUBE PLUGGING, TUBE PULLING, WELD REPAIR, AND HYDROSTATIC LEAK TESTING.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

POINT BEACH 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Wisconsin/Manitowoc 15 miles N of Manitowoc
2.	Electric Reliability Council	Mid-America Interpool Network
3.	Type of Reactor	PWR
4.	Power Level (MWT)	1518
5.	Design Electrical Rating (MWE Net)	497
6.	Date of Initial Criticality	May 30, 1972
7.	Date of Commercial Operation	October 1, 1972
8.	Condenser Cooling Method	Straight Through
9.	Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10.	Licensee	Wisconsin Electric Power Co.
11.	Corporate Address	231 West Michigan Milwaukee, Wisconsin
12.	Corporate Contact	S. Burstein
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	R. F. Warnick
19.	IE Resident Inspector	J. D. SMITH
20.	Licensing Project Manager	C. M. Trammell
21.	Docket Number	50-301
22.	License Number/Date of Issue	DPR-27 issued 11/16/71
23.	Public Document Room	Document Department, Library University of Wisconsin-Stevens Point Stevens Point, Wisconsin 54481 Manitowoc Public Library Manitowoc, Wisconsin (Temporary LPDR for spent fuel pool capacity increase amendment only

P-12

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

+ The unit was taken offline on April 11, 1980 for refueling and maintenance.

+LAST IE SITE INSPECTION DATE: March 6 and 20, 1980

+ INSPECTION REPORT NO: 80-04

SUMMARY

INSPECTION SUMMARY

+ Inspection on February 25-26, (80-02): Licensee actions relative to IE Bulletin No. 79-14, including general discussion on NRC requirements, work procedure review, and review of analysis and calculations. The inspection involved 16 inspector hours onsite by one NRC inspector. No items of noncompliance or deviations were identified.

+ Inspection on February 19-20, (80-03): Routine, unannounced inspection to review plant operations, non-routine event follow-up and independent inspection. The inspection involved 15 inspector hours onsite by one NRC inspector. Of the three areas inspected, no items of noncompliance were identified in two areas. One item of noncompliance was identified in one area (minimum number of Auxiliary Feedwater Pumps not capable of automatic operation during two unit operation).

+ Inspection on March 6 and 20, (80-04): QA/QC programs, implementing procedures, work activities, and records relative to the steam generators tube inspections. The inspection involved six inspector hours onsite by one NRC inspector. No items of noncompliance or deviations were identified.

REPORTS RECEIVED FROM LICENSEE

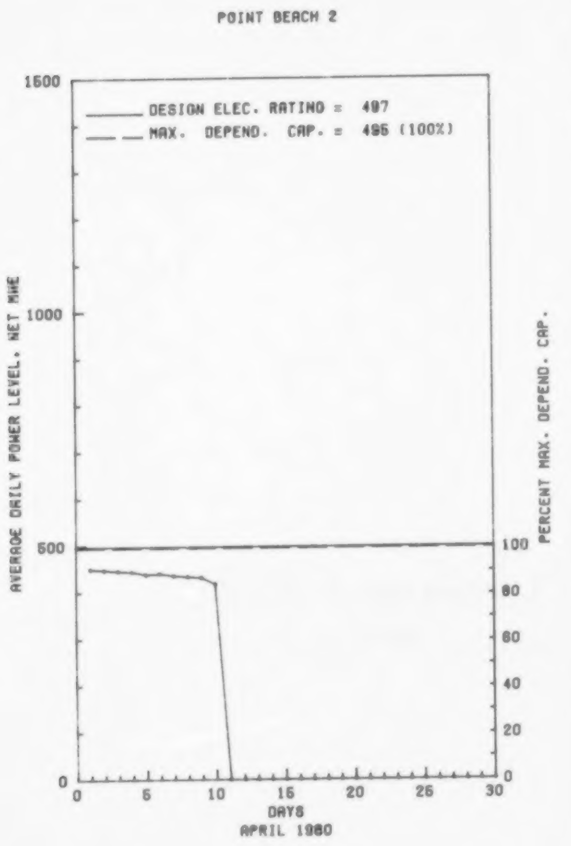
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-03/ 01T-0	04/08/80	04/17/80	Pressurizer level channel failed low.

REVIEWED BY OMPA LOEB: T. C. Cintula **DATE:** 6·12·80

REVIEWED BY NRR DOR: C M Trammell **DATE:** 6/13/80

POINT BEACH 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-301
2. Utility Contact: C.W. FAY (414) 277-2811
3. Licensed Thermal Power (MWt): 1518
4. Nameplate Rating (Gross MWe): 582 x 0.9 = 524
5. Design Electrical Rating (Net MWe): 497
6. Maximum Dependable Capacity (Gross MWe): 519
7. Maximum Dependable Capacity (Net MWe): 495
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

ITEM 21 CALCULATED BY USING A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	719.0	2,903.0	61,920.0
12. Hours Reactor Was Critical	246.8	2,088.2	60,985.1
13. Reactor Reserve Shutdown Hours	9.4	9.4	175.5
14. Hours Generator On-Line	240.6	2,073.0	59,800.0
15. Unit Reserve Shutdown Hours	2.0	3.1	109.5
16. Gross Thermal Energy (MWH)	328,351	3,027,064	80,679,690
17. Gross Electrical Energy (MWH)	110,200	1,004,670	27,366,010
18. Net Electrical Energy (MWH)	103,713	956,893	26,032,020
19. Unit Service Factor	33.5	71.4	88.0
20. Unit Availability Factor	33.7	71.5	88.2
*21. Unit Capacity Factor (MDC Net)	29.1	66.6	78.2
22. Unit Capacity Factor (DER Net)	29.0	66.3	77.1
23. Unit Forced Outage Rate	.0	14.5	2.0

24. Shutdowns Scheduled Over Next 6 Months (Type, Date and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 05/15/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type^1	Duration (Hours)	Reason^2	Method of Shutting Down Reactor^3	Licensee Event Report No.	System Code^4	Component Code^5	Cause & Corrective Action to Prevent Recurrence
1	04/11/80	S	478.4	C	1		RC	FUELXX	REFUELING OUTAGE.

SUMMARY

THE UNIT WAS SHUTDOWN ON 04/11/80 FOR REFUELING.

LEGEND

^{1}Type:
F–Forced
S–Scheduled

^{2}Reason:
A–Equipment Failure
B–Maintenance or Test
C–Refueling
D–Regulatory Restrictio
E–Operator Training & License Examination
F–Administrative
G–Operational Error
H–Other

^{3}Method:
1–Manual
2–Manual Scram
3–Automatic Scram
4–Continuations
5–Load Reduction
9–Other

^{4}Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

^{5}Exhibit H:
Same source.

PRAIRIE ISLAND 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Minnesota/Goodhue 28 miles SE of Minneapolis
2.	Electric Reliability Council	Mid-Continent Area Reliability Coordination Agreement
3.	Type of Reactor	PWR
4.	Power Level (MWT)	1650
5.	Design Electrical Rating (MWE Net)	530
6.	Date of Initial Criticality	December 1, 1973
7.	Date of Commercial Operation	December 16, 1973
8.	Condenser Cooling Method	Cooling Towers
9.	Condenser Cooling Water Source	Mississippi River

Utility & Contractor Information

10.	Licensee	Northern States Power Company
11.	Corporate Address	414 Nicollet Mall - 8th Floor Minneapolis, Minnesota 55401
12.	Corporate Contact	G. H. Neils (Oper.) L. O. Mayer (Lic.)
13.	Architect/Engineer	Pioneer Services & Eng. Co.
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Northern States Power Co.

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	D. C. BOYD
19.	IE Resident Inspector	C. D. Feierabend 612-388-8209
20.	Licensing Project Manager	R. Martin
21.	Docket Number	50-282
22.	License Number/Date of Issue	DPR-42 issued 8/9/73
23.	Public Document Room	Environmental Conservation Library Minneapolis Public Library 300 Nicollet Mall Minneapolis, Minnesota 55401

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

Operating routinely.

*LAST IE SITE INSPECTION DATE: March 1-31, 1980

* INSPECTION REPORT NO: 80-05

SUMMARY

INSPECTION SUMMARY

* Inspection on February 1-29, (80-03): Management meeting on February 26, 1980. Routine resident inspection of plant operation, maintenance, surveillance, security, radioactive waste system, and followup of operating event. The inspection involved 88 inspector hours onsite by one NRC inspector. No items of noncompliance or deviations were identified.

* Inspection on March 1-31, (80-05): Routine resident inspection of plant operation, maintenance, followup of licensee reported events and review of licensee small break loss of coolant accident procedures. The inspection involved 85 resident inspection hours onsite and 24 inspector hours onsite by one regional based inspector. No items of noncompliance or deviations were identified.

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-09/ 03L-0	03/10/80	04/09/80	During normal operation, Nuclear Power Range Channel 1N44 was inoperable for a few minutes.
80-10/ 03L-0	03/11/80	04/10/80	During surveillance test, No. 11 containment vacuum breaker isolation valve opened at 0.57 psi, this is outside TS 4.4.C.

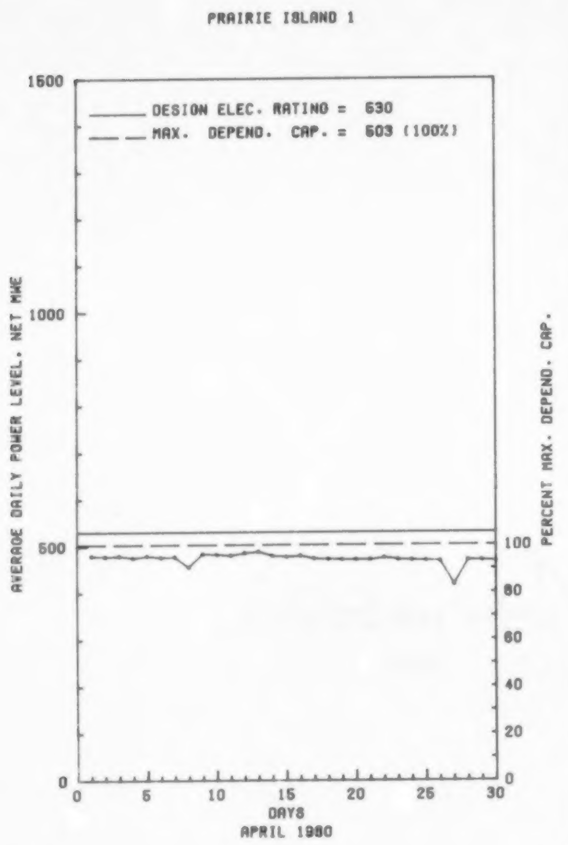
==

REVIEWED BY OMPA LOEB: T. C. Cintula DATE: 6•12•80

REVIEWED BY NRR DOR: [illegible] DATE: [illegible]

PRAIRIE ISLAND 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-282
2. Utility Contact: A. A. Hunstad (612) 388-4767
3. Licensed Thermal Power (MWt): 1650
4. Nameplate Rating (Gross MWe): 659 x 0.9 = 593
5. Design Electrical Rating (Net MWe): 530
6. Maximum Dependable Capacity (Gross MWe): 534
7. Maximum Dependable Capacity (Net MWe): 503
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	719.0	2,903.0	55,871.0
12. Hours Reactor Was Critical	719.0	2,771.9	43,980.3
13. Reactor Reserve Shutdown Hours	.0	.0	5,531.9
14. Hours Generator On-Line	719.0	2,766.8	42,890.4
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,175,683	4,504,122	66,583,116
17. Gross Electrical Energy (MWH)	362,190	1,410,680	21,564,350
18. Net Electrical Energy (MWH)	340,894	1,323,602	20,162,854
19. Unit Service Factor	100.0	95.3	76.8
20. Unit Availability Factor	100.0	95.3	76.8
*21. Unit Capacity Factor (MDC Net)	94.0	90.6	71.7
22. Unit Capacity Factor (DER Net)	89.2	86.0	68.1
23. Unit Forced Outage Rate	.0	.0	12.8

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each)
REFUELING, AUGUST 4, 1980, 8 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	NONE								

SUMMARY

OPERATION WAS LOAD FOLLOWING THROUGHOUT THE REPORT PERIOD, WITH NO SHUTDOWNS OR REDUCTIONS REPORTED.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

PRAIRIE ISLAND 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Minnesota/Goodhue 28 miles SE of Minneapolis
2.	Electric Reliability Council	Mid-Continent Area Reliability Coordination Agreement
3.	Type of Reactor	PWR
4.	Power Level (MWT)	1650
5.	Design Electrical Rating (MWE Net)	530
6.	Date of Initial Criticality	December 17, 1974
7.	Date of Commercial Operation	December 21, 1974
8.	Condenser Cooling Method	Cooling Towers
9.	Condenser Cooling Water Source	Mississippi River

Utility & Contractor Information

10.	Licensee	Northern States Power Company
11.	Corporate Address	414 Nicollet Mall - 8th Floor Minneapolis, Minnesota 55401
12.	Corporate Contact	G. H. Neils (Oper.) L. O. Mayer (Lic.)
13.	Architect/Engineer	Pioneer Services & Eng. Co.
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Northern States Power Co.

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	D. C. BOYD
19.	IE Resident Inspector	C. D. Feierabend 612-388-8209
20.	Licensing Project Manager	R. Martin
21.	Docket Number	50-306
22.	License Number/Date of Issue	DPR-60 issued 10/29/74
23.	Public Document Room	Environmental Conservation Library Minneapolis Public Library 300 Nicollet Mall Minneapolis, Minnesota 55401

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

Operating routinely.

*LAST IE SITE INSPECTION DATE: March 1-31, 1980

* INSPECTION REPORT NO: 80-06

SUMMARY

INSPECTION SUMMARY

* Inspection on February 1-29, (80-04): Management meeting on February 26, 1980. Routine resident inspection of plant operation, maintenance, surveillance, security, radioactive waste system, and followup of operating event. The inspection involved 88 inspector hours onsite by one NRC inspector. No items of noncompliance or deviations were identified.

* Inspection on March 1-31, (80-06): Routine resident inspection of plant operation, maintenance, followup of licensee reported events and review of licensee small break loss of coolant accident procedures. The inspection involved 85 resident inspection hours onsite and 24 inspector hours onsite by one regional based inspector. No items of noncompliance or deviations were identified.

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-11/ 03L-0	03/12/80	04/11/80	With one charging pump out of service, No. 22 charging pump was started but its motor breaker would not stay closed and the pump stopped.

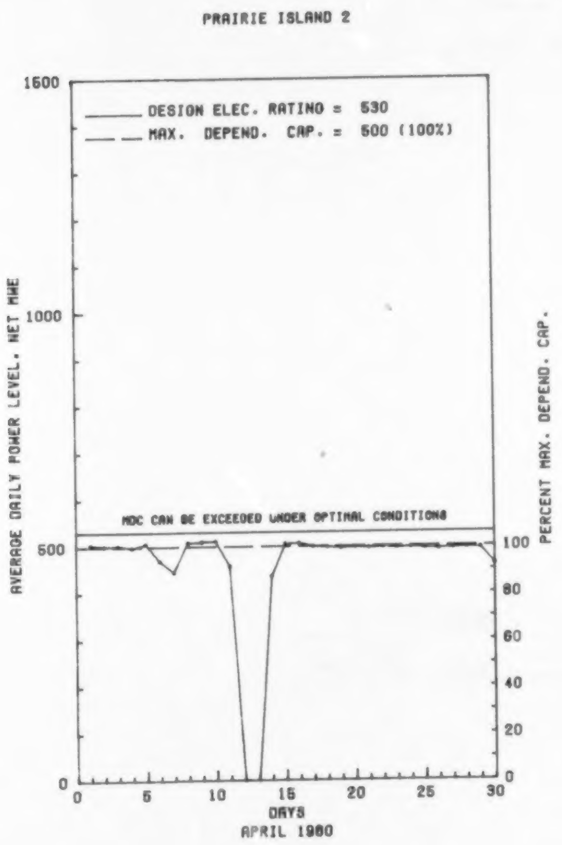
REVIEWED BY OMPA LOEB: T.C. Cintula DATE: 6.12.80

REVIEWED BY NRR DOR: R E Martin DATE: 6/13/80

GPO 875-887

PRAIRIE ISLAND 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-306
2. Utility Contact: A. A. Hunstad (612) 388-4767
3. Licensed Thermal Power (MWt): 1650
4. Nameplate Rating (Gross MWe): 659 x 0.9 = 593
5. Design Electrical Rating (Net MWe): 530
6. Maximum Dependable Capacity (Gross MWe): 531
7. Maximum Dependable Capacity (Net MWe): 500
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	719.0	2,903.0	46,989.0
12. Hours Reactor Was Critical	676.6	1,731.9	40,860.9
13. Reactor Reserve Shutdown Hours	42.4	42.4	1,448.4
14. Hours Generator On-Line	671.0	1,684.5	40,085.5
15. Unit Reserve Shutdown Hours	.0	0	.0
16. Gross Thermal Energy (MWH)	1,089,161	2,639,898	62,312,401
17. Gross Electrical Energy (MWH)	350,520	860,920	19,915,050
18. Net Electrical Energy (MWH)	329,176	802,396	18,641,028
19. Unit Service Factor	93.3	58.0	85.3
20. Unit Availability Factor	93.3	58.0	85.3
*21. Unit Capacity Factor (MDC Net)	91.6	55.3	79.3
22. Unit Capacity Factor (DER Net)	86.4	52.2	74.9
23. Unit Forced Outage Rate	.0	.6	4.6

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
: REFUELING, FEBRUARY 1981, 6 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	04/11/80	S	48.0	B	1				SHUTDOWN TO MAKE REPAIRS TO TURBINE OIL SYSTEM ANL' OTHER MAINTENANCE.

SUMMARY

OPERATION WAS LOAD FOLLOWING DURING THE REPORT PERIOD, WITH AN OUTAGE FROM 04/11/80 TO 04/13/80 TO REPAIR THE TURBINE OIL SYSTEM TO PERFORM MINOR MAINTENANCE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

QUAD CITIES 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1. Location (state/county) (nearest population center)	Illinois/Rock Island 20 miles NE of Moline
2. Electric Reliability Council	Mid-America Interpool Network
3. Type of Reactor	BWR
4. Power Level (MWT)	2511
5. Design Electrical Rating (MWE Net)	789
6. Date of Initial Criticality	October 18, 1971
7. Date of Commercial Operation	February 18, 1973
8. Condenser Cooling Method	Spray Canal
9. Condenser Cooling Water Source	Mississippi River

Utility & Contractor Information

10. Licensee	Commonwealth Edison Co.
11. Corporate Address	P. O. Box 767 Chicago, Illinois 60690
12. Corporate Contact	D. L. Peoples
13. Architect/Engineer	Sargent and Lundy
14. Nuclear Steam Supply System	GE
15. Constructor	United Engineers & Constructors, Inc.

Regulatory Information

16. IE Region responsible	III
17. IE Branch Chief	R. Heishman
18. IE Section Chief	R. Spessard
19. IE Resident Inspector	N. J. Chrissotimus (309) 654-2227
20. Licensing Project Manager	R. B. Bevan
21. Docket Number	50-254
22. License Number/Date of Issue	DPR-29 issued 10/1/71
23. Public Document Room	Molina Public Library 504 17th Street Moline, Illinois 61265

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

MANAGERIAL ITEMS

+ A public news conference was held on February 5, 1980 in the Moline Holiday Inn to discuss recent events at Quad-Cities Station. The Regional Director was accompanied by the Public Affairs Officer and other NRC representatives. (80-04)

PLANT STATUS

Unit operating routinely.

+LAST IE SITE INSPECTION DATE: January 4; March 6-7 and 9-11, 1980

+ INSPECTION REPORT NO: 80-08

SUMMARY

INSPECTION SUMMARY

+ Inspection on February 4 through March 3, (80-04): Routine, unannounced resident inspection of maintenance, plant operations, physical protection-security organization; physical barriers; and access control (identification, authorization, badging, search, and escorting); review and followup on licensee event reports, refueling activities, surveillance of safety related systems, followup on Headquarters request, radiation training, IE Circular followup, independent inspection and media contacts. The inspection involved 147 inspector hours onsite by one NRC inspector. Of the 13 areas inspected, no apparent items of noncompliance were identified in 11 areas; 3 items of noncompliance were identified in two areas (exceeding a limiting condition for operation; failure to follow procedures; and inadequate procedure).

+ Inspection on February 27-28, (80-05): Licensee actions relative to IE Bulletin No. 79-14 including general discussion on NRC requirements, work procedure review and review of analysis and calculations. The inspection involved 12 inspector hours onsite by one NRC inspector. No items of noncompliance or deviations were identified.

+ Inspection on March 4-25, (80-06): Routine, unannounced inspection consisting of operations safety verification, maintenance, surveillance observation, IE Bulletin followup, licensee event followup, followup on headquarters requests, design changes and modifications, review and audits, and independent inspection. The inspection involved 189 inspector hours onsite by three NRC inspectors. Of the nine areas inspected, no items of noncompliance were identified.

+ Inspection on March 25-27, (80-07): Routine, unannounced inspection of Environmental Protection program including management controls; implementation of environmental monitoring program (radiological and non-radiological); and Confirmatory Measurements of radiological effluents, including comparison of licensee result of sample split in November 1979 to those of the NRC Reference Laboratory and collection of radioactive effluent samples for future comparison. The inspection involved 36 inspector hours onsite by two NRC inspectors. For the areas inspected, no items of noncompliance or deviations were identified.

+ Inspection on January 4; March 6-7 and 9-11, (80-08): Routine, announced inspection of the containment integrated leak rate test; review of previous Unresolved Items. The inspection involved 186 inspector hours onsite by three NRC inspectors. Of the two areas inspected, one item of noncompliance was identified in one area (failure to follow procedure).

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-05/03L-0	02/24/80	03/20/80	During MSIV Closure Timing, valve AO-1-203-1B failed to close when the control switch was operated.
80-07/03L-0	03/08/80	03/25/80	The B main steam line radiation monitor was found to be reading low.
80-08/03L-0	03/08/80	03/27/80	After having performed monthly preventive maintenance on the 1/2 diesel, it would not start from the control room.

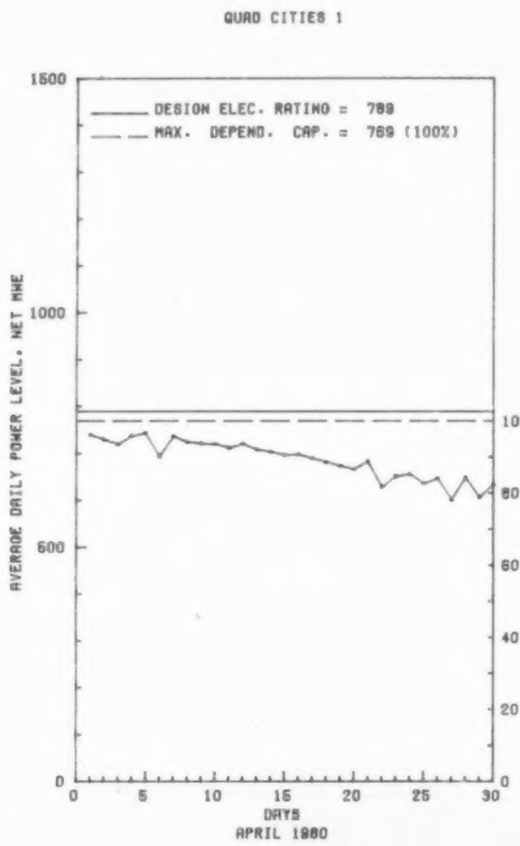
==

REVIEWED BY OMPA LOEB: Bobe **DATE:** 6/12/80

REVIEWED BY NRR DOR: RBBevan **DATE:** 6/13/80

QUAD CITIES 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-254
2. Utility Contact: D. Hannum (309) 654-2241
3. Licensed Thermal Power (MWt): 2511
4. Nameplate Rating (Gross MWe): 920 x 0.9 = 828
5. Design Electrical Rating (Net MWe): 789
6. Maximum Dependable Capacity (Gross MWe): 813
7. Maximum Dependable Capacity (Net MWe): 769
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	719.0	2,903.0	69,863.0
12. Hours Reactor Was Critical	719.0	2,903.0	57,617.1
13. Reactor Reserve Shutdown Hours	.0	.0	3,421.9
14. Hours Generator On-Line	719.0	2,903.0	54,963.9
15. Unit Reserve Shutdown Hours	.0	.0	909.2
16. Gross Thermal Energy (MWH)	1,652,783	6,642,019	111,186,000
17. Gross Electrical Energy (MWH)	535,624	2,173,081	35,796,135
18. Net Electrical Energy (MWH)	494,512	2,051,141	33,466,677
19. Unit Service Factor	100.0	100.0	78.6
20. Unit Availability Factor	100.0	100.0	79.9
21. Unit Capacity Factor (MDC Net)	89.4	91.9	62.3
22. Unit Capacity Factor (DER Net)	87.2	89.6	60.7
23. Unit Forced Outage Rate	.0	.0	7.4

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
6	04/29/80	F	.8	A	5	80-12/03L	CB	PUMPXX	1A RECIRC PUMP TRIPPED ON LOW OIL PRESSURE WHEN 1A1 RECIRC M-G SET OIL PUMP TRIPPED; 1B RECIRC PUMP WAS REDUCED TO 30 PERCENT POWER. OIL PUMP WAS RESTARTED AND PERFORMED SATISFACTORILY; 1A RECIRC PUMP WAS RESTARTED AND PERFORMED NORMALLY.

SUMMARY

OPERATION WAS ROUTINE DURING REPORT PERIOD, EXCEPT FOR A RECIRCULATION PUMP TRIP DUE TO LOW OIL PRESSURE WHEN A MOTOR-GENERATOR SET OIL PUMP TRIPPED. OTHER BRIEF REDUCTIONS WERE REQUIRED FOR WEEKLY TURBINE-GENERATOR TESTING AND REVERSAL OF MAIN CONDENSER FLOW.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

QUAD CITIES 2

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Illinois/Rock Island 20 miles NE of Moline
2.	Electric Reliability Council	Mid-America Interpool Network
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2511
5.	Design Electrical Rating (MWE Net)	789
6.	Date of Initial Criticality	April 26, 1972
7.	Date of Commercial Operation	March 10, 1973
8.	Condenser Cooling Method	Spray Canal
9.	Condenser Cooling Water Source	Mississippi River

Utility & Contractor Information

10.	Licensee	Commonwealth Edison Co.
11.	Corporate Address	P. O. Box 767 Chicago, Illinois 60690
12.	Corporate Contact	D. L. Peoples
13.	Architect/Engineer	Sargent & Lundy
14.	Nuclear Steam Supply System	GE
15.	Constructor	United Engineers & Constructors, Inc.

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	R. Spessard
19.	IE Resident Inspector	N. J. Chrissotimus (309) 654-2227
20.	Licensing Project Manager	R. B. Bevan
21.	Docket Number	50-265
22.	License Number/Date o. Issue	DPR-30 issued 3/21/72
23.	Public Document Room	Moline Public Library 504 17th Street Moline, Illinois 61265

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENTS

- While performing a Reactor Vessel Pressure test on Unit 2, a visual inspection revealed water drops on the "A" loop core spray piping. Ultrasonic inspection disclosed several crack indications located in the pipe to elbow welds between the Reactor Vessel and first manual location valve in both core spray loops. (80-09)

MANAGERIAL ITEMS

- A public news conference was held on February 5, 1980 in the Moline Holiday Inn to discuss recent events at Quad-Cities Station. The Regional Director was accompanied by the Public Affairs Officer and other NRC representatives. (80-06)

PLANT STATUS

- The unit was returned to service on April 21, 1980 following a refueling/maintenance outage which began on November 25, 1979, and it is operating routinely.

*LAST IE SITE INSPECTION DATE: March 25-27, 1980

- INSPECTION REPORT NO: 80-10

SUMMARY

INSPECTION SUMMARY

- Inspection on January 4; March 6-7 and 9-11, (80-01): Routine, announced inspection of the containment integrated leak rate test; review of previous Unresolved Items. The inspection involved 186 inspector hours onsite by three NRC inspectors. Of the two areas inspected, one item of noncompliance was identified in one area (failure to follow procedure).

- Inspection on February 4 through March 3, (80-06): Routine, unannounced resident inspection of maintenance, plant operations, physical protection-security organization; physical barriers; and access control (identification, authorization, badging, search, and escorting); review and followup on licensee event reports, refueling activities, surveillance of safety related systems, followup on Headquarters request, radiation training, IE Circular followup, independent inspection and media contacts. The inspection involved 147 inspector hours onsite by one NRC inspector. Of the 13 areas inspected, no apparent items of noncompliance were identified in 11 areas; two items of noncompliance were identified in one area (exceeding a limiting condition for operation).

- Inspection on February 27-28, (80-08): Licensee actions relative to IE Bulletin No. 79-14 including general discussion on NRC requirements, work procedure review and review of analysis and calculations. The inspection involved 12 inspector hours onsite by one NRC inspector. No items of noncompliance or deviations were identified.

- Inspection on March 4-25, (80-09): Routine, unannounced inspection consisting of operations safety verification, maintenance, surveillance observation, IE Bulletin followup, licensee event followup, followup on headquarters requests, design changes and modifications, review and audits, and independent inspection. The inspection involved 189 inspector hours onsite by three NRC inspectors. Of the nine areas inspected, no items of noncompliance were identified.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-05/ 03L-0	03/03/80	03/24/80	The five second delay timer for 1002B RHR pump, failed to actuate relay K21A and the relay failed to energize.
80-06/ 01T-0	03/08/80	03/22/80	Discovery of indications in core spray piping.
80-07/ 03L-0	03/16/80	04/09/80	A width-wise crack indication along the upper machined surface located near the beam bolt on jet pump No. 16 was discovered.
80-08/ 03L-0	03/05/80	03/26/80	Two five-second delay timers failed to actuate the timer contacts of the 1002D RHR pump.

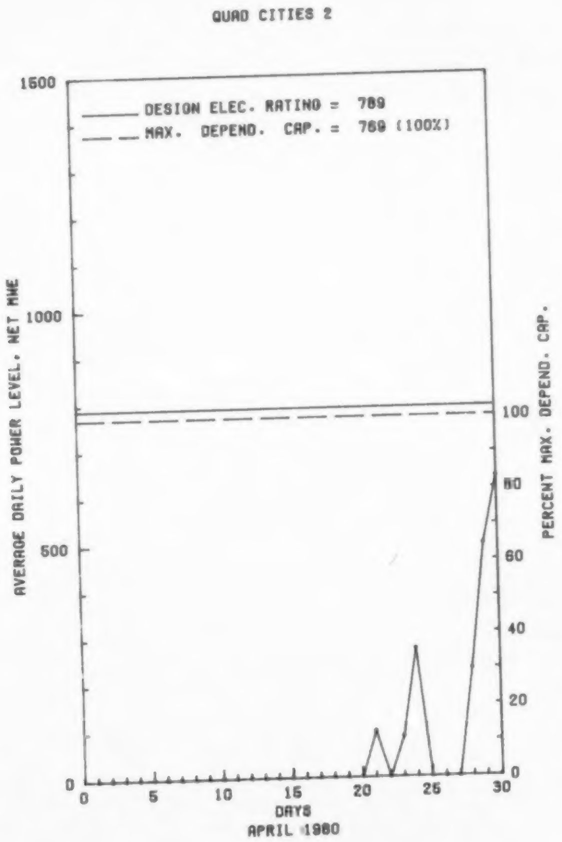
- Inspection on March 25-27, (80-10): Routine, unannounced inspection of Environmental Protection program including management controls; implementation of environmental monitoring program (radiological and non-radiological); and Confirmatory Measurements of radiological effluents, including comparison of licensee result of sample split in November 1979 to those of the NRC Reference Laboratory and collection of radioactive effluent samples for future comparison. The inspection involved 36 inspector hours onsite by two NRC inspectors. For the areas inspected, no items of noncompliance or deviations were identified.

REVIEWED BY OMPA LOEB: J E Bobe DATE: 6/12/80

REVIEWED BY NRR DOR: RBBevan DATE: 6/13/80

QUAD CITIES 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-265
2. Utility Contact: D. Mannum (309) 654-2241
3. Licensed Thermal Power (MWt): 2511
4. Nameplate Rating (Gross MWe): 920 x 0.9 = 828
5. Design Electrical Rating (Net MWe): 789
6. Maximum Dependable Capacity (Gross MWe): 813
7. Maximum Dependable Capacity (Net MWe): 769
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	719.0	2,903.0	68,973.0
12. Hours Reactor Was Critical	177.3	177.3	53,306.2
13. Reactor Reserve Shutdown Hours	.0	.0	2,985.8
14. Hours Generator On-Line	119.7	119.7	50,911.5
15. Unit Reserve Shutdown Hours	.0	.0	702.9
16. Gross Thermal Energy (MWH)	167,344	167,344	103,183,453
17. Gross Electrical Energy (MWH)	46,936	46,985	32,910,035
18. Net Electrical Energy (MWH)	39,394	1,728	30,744,253
19. Unit Service Factor	16.6	4.1	73.8
20. Unit Availability Factor	16.6	4.1	74.8
21. Unit Capacity Factor (MDC Net)	7.1	.1	58.0
22. Unit Capacity Factor (DER Net)	6.9	.1	56.5
23. Unit Forced Outage Rate	49.5	49.5	9.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
13	11/25/79	S	481.8	C	4				UNIT TWO CYCLE 4 REFUELING OUTAGE CONTINUES.
1	04/20/80	F	39.4	A	1	80-11/03L	CC	VALVEX	REPAIR 3C ELECTROMATIC RELIEF VALVE.
2	04/22/80	F	78.1	A	1	80-10/03L	SE	VESSEL	REPAIR PRESSURE SUPPRESSION CHAMBER VACUUM BREAKERS.

SUMMARY

THE UNIT COMPLETED ITS REFUELING OUTAGE AND RESUMED POWER OPERATIONS ON APRIL 21, 1980.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

RANCHO SECO

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	California/Sacramento 25 miles SE of Sacramento
2.	Electric Reliability Council	Western Systems Coordinating Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2772
5.	Design Electrical Rating (MWE Net)	918
6.	Date of Initial Criticality	September 16, 1974
7.	Date of Commercial Operation	April 17, 1975 (92.6% of full power)
8.	Condenser Cooling Method	Closed Cycle
9.	Condenser Cooling Water Source	Natural Draft Cooling Towers - Folsom Canal

Utility & Contractor Information

10.	Licensee	Sacramento Municipal Utility District
11.	Corporate Address	6201 South Street P. O. Box 15830 Sacramento, California
12.	Corporate Contact	J. J. Mattimoe Assistant General Manager, Chief Engineer
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	B&W
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	V
17.	IE Branch Chief	J. L. Crews
18.	IE Section Chief	B. H. Faulkenberry
19.	IE Resident Inspector	H. L. Canter (209) 748-2791
20.	Licensing Project Manager	D. J. Garner
21.	Docket Number	50-312
22.	License Number/Date of Issue	DPR-54, 8/16/74
23.	Public Document Room	Business and Municipal Department Sacramento City-County Library 828 I Street Sacramento, California 95814

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO NRC'S EXEMPTION TO 10 CFR 50, PARAGRAPH 50.46(A), DATED DECEMBER 15, 1978, PARAGRAPH IV(1), AND WITHOUT IMPLEMENTATION OF THE MODIFICATIONS, TWO VALVES ON THE HPI CROSS-CONNECT LINE WERE CLOSED DURING THE PERIOD OF DECEMBER 17 THROUGH DECEMBER 27, 1979. THE TWO VALVES WERE CLOSED AS THE RESULT OF THE LICENSEE FAILING TO ADEQUATELY IMPLEMENT PROCEDURE (A.15) FOR OPERATING THE MAKEUP, PURIFICATION AND LETDOWN SYSTEM. DURING THIS ENTIRE PERIOD OF TIME, THE REACTOR WAS BEING OPERATED AT LEVELS ABOVE 90% OF RATED POWER.
(8006 VIOLATION)

CONTRARY TO TS 3.3.1, THE REACTOR REMAINED CRITICAL BETWEEN DECEMBER 17, 1979 AND JANUARY 9, 1980 (EXCEPT FOR A 16 HOUR SHUTDOWN) WITH ONLY ONE OF THE THREE HIGH PRESSURE INJECTION PUMPS OPERABLE. THE MAKEUP PUMP THAT WAS BEING USED AT THE TIME AS A HPI PUMP WAS INOPERABLE AS AN HPI PUMP BECAUSE MANUAL VALVES IN THE NUCLEAR SERVICE RAW WATER SUCTION AND DISCHARGE LINES FOR THE MAKEUP PUMP LUBE OIL AND ROOM AIR COOLERS WERE CLOSED.
(8006 INFRACTION)

CONTRARY TO TS 6.8.1 AND PROCEDURE A15, DEALING WITH RESTORING THE "B" HIGH PRESSURE INJECTION PUMP TO SERVICE, ON JANUARY 9, 1980, WHEN THE "B" HIGH PRESSURE INJECTION PUMP WAS RETURNED TO SERVICE AND THE MAKEUP PUMP WAS REMOVED FROM SERVICE AS A HIGH PRESSURE INJECTION PUMP, THE OPERATOR FAILED TO CHANGE THE POWER SUPPLY FROM 52-23166 TO 52-24172,THEREBY LEAVING SFV-23508 WITHOUT ELECTRICAL POWER AND INCAPABLE OF CLOSING AS REQUIRED UPON INITIATION OF A SAFETY FEATURES ACTUATION SIGNAL. THIS CONDITION WAS CORRECTED ON JANUARY 10, 1980.
(8006 INFRACTION)

INSPECTION STATUS

SYSTEMS AND COMPONENT PROBLEMS:

- Fire protection and small break loca modifications completed.

FACILITY ITEMS (PLANS AND PROCEDURES):

None.

MANAGERIAL ITEMS:

- Hearing on NRC Order of May 7, 1979 completed receiving testimony on May 14, 1980.

PLANT STATUS:

- Plant continued refueling throughout the month of April.

LAST IE SITE INSPECTION DATE: 04/14-26 & 05/05-09/80+

INSPECTION REPORT NO. 50-312/80-15+

SUMMARY

INSPECTION SUMMARY

- Inspection on April 7-11, 1980 (Report No. 50-312/80-09)
  Areas Inspected: Routine, unannounced inspection of the status of completion of selected fire protection modifications; status of completion of other required modifications; changes in the onsite organization or staffing; the program for receipt, storage and handling of equipment and materials; followup on IE Bulletins and Circulars; and followup on previous inspection findings. The inspection involved 36 inspector hours onsite by one NRC inspector.

  Results: No items of noncompliance or deviations were identified.

NRC POWER RESTRICTIONS: None

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-15/ 01T-0	03-12-80	03-25-80	Clad degradation on fuel assy 1c47.
80-16/ 03L-0	03-10-80	04-03-80	As result of Bulletin 79-14, three pipe supports required modification.
80-17/ 03L-0	03-22-80	04-10-80	Limitork mod SMB-2 Opr on SF vlv inoperable due to excess wear.
80-18/ 03L-0	03-24-80	04-11-80	SFV-72502 inoperable due to wire pulled loose from terminal lug.
80-23/ 01T-0	04-14-80	04-15-80	Dresser porv and code safety valves only designed for sat steam.

- Inspection on March 3-31, 1980 (Report No. 0-312/80-10)
  Areas Inspected: Routine inspections of Licensee Event Report followup; followup on items of noncompliance; and independent inspection effort. The inspection involved 52 inspector hours by the NRC Senior Resident Inspector.

  Results: Of the three areas inspected, no items of noncompliance or deviations were found in two areas. One item of noncompliance was found in one area (Infraction - Movement of objects over the fuel transfer canal without approved procedures).

- Inspection on April 7-10, 1980 (Report No. 50-312/80-11)
  Areas Inspected: Included a review of vital area physical barriers and detection aids. The inspectors also reviewed the status of construction items and plans for new security hardware which upgrade site security features.

  Results: No items of noncompliance or deviations were identified.

- Inspection on April 7-8 & 10, 1980 (Report No. 50-312/80-12)
  Areas Inspected: Routine, announced inspection by a Regional based inspector of the licensee's activities performed in response to IE Bulletin 79-01B, "Environmental Qualification of Class IE Equipment." The inspection involved 18 inspector hours onsite by one NRC inspector.

  Results: No items of noncompliance or deviations were identified.

- Inspection on May 5-9, 1980 (Report No. 50-312/80-13)
  Report being prepared; to be reported next month.

- Inspection on March 25-27, 1980 (Report No. 50-312/80-14)
  Areas Inspected: Routine, unannounced inspection by Regional based inspectors of the licensee's activities performed in response to IE Bulletin 79-02, "Pipe Support Base Plate Design Using Concrete Expansion Anchor Bolts;" IE Bulletin 79-13, "Cracking in Feedwater System Piping;" and an examination of a reported weld cracking problem during valve installation. The inspection involved 24 inspector hours onsite by one NRC inspector.

  Results: No items of noncompliance or deviations were identified.

- Inspection on April 14-26 & May 5-9, 1980 (Report No. 50-312/80-15)
  Report being prepared; to be reported next month.

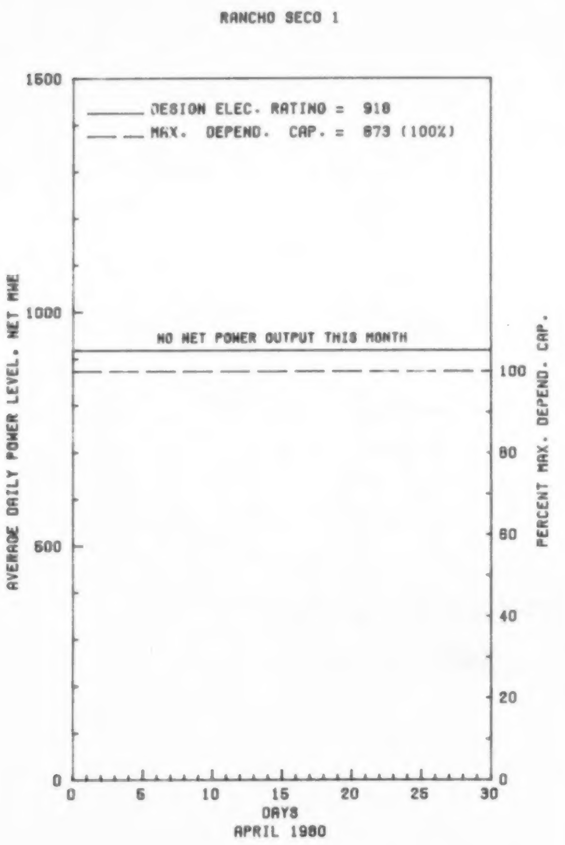
- Inspection on April 1-30, 1980 (Report No. 50-312/80-16)
  Report being prepared; to be reported next month.

REVIEWED BY OMPA LOEB: **DATE:** 6/12/80

REVIEWED BY NRR DOR: **DATE:** 6/13/80

RANCHO SECO

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-312
2. Utility Contact: R. W. Colombo (916) 452-3211
3. Licensed Thermal Power (MWt): 2772
4. Nameplate Rating (Gross MWe): 1070 x 0.9 = 963
5. Design Electrical Rating (Net MWe): 918
6. Maximum Dependable Capacity (Gross MWe): 917
7. Maximum Dependable Capacity (Net MWe): 873
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	719.0	2,903.0	45,160.0
12. Hours Reactor Was Critical	.0	271.0	26,344.5
13. Reactor Reserve Shutdown Hours	.0	.0	3,975.1
14. Hours Generator On-Line	.0	266.9	25,208.2
15. Unit Reserve Shutdown Hours	.0	.0	1,210.2
16. Gross Thermal Energy (MWH)	0	689,820	63,673,838
17. Gross Electrical Energy (MWH)	0	221,784	21,506,320
18. Net Electrical Energy (MWH)	0	206,442	20,317,332
19. Unit Service Factor	.0	9.2	57.1
20. Unit Availability Factor	.0	9.2	59.8
21. Unit Capacity Factor (MDC Net)	.0	8.1	52.7
22. Unit Capacity Factor (DER Net)	.0	7.7	50.1
23. Unit Forced Outage Rate	.0	14.5	32.0

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 05/15/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
02	01/14/80	S	719.0	C	4				REFUELING OUTAGE CONTINUES.

SUMMARY

THE UNIT REMAINED SHUTDOWN THROUGHOUT THE REPORT PERIOD FOR REFUELING.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

ROBINSON 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	South Carolina/Darlington 5miles NW of Hartsville
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2300
5.	Design Electrical Rating (MWE Net)	700
6.	Date of Initial Criticality	September 20, 1970
7.	Date of Commercial Operation	March 7, 1971
8.	Condenser Cooling Method	Recirculation from impounded lake
9.	Condenser Cooling Water Source	Robinson Impoundment

Utility & Contractor Information

10.	Licensee	Carolina Power and Light Company
11.	Corporate Address	411 Fayetteville Street Raleigh, North Carolina
12.	Corporate Contact	J. A. Jones Executive Vice President
13.	Architect/Engineer	Ebasco
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Ebasco

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R.C. Lewis
18.	IE Section Chief	P. J. Kellogg
19.	IE Resident Inspector	R. Sauer
20.	Licensing Project Manager	D. Neighbors
21.	Docket Number	50-261
22.	License Number/Date of Issue	DPR-23, 7/31/70 (5 Mwt), Full power authorization issued 9/23/70
23.	Public Document Room	Hartsville Memorial Library Home and Fifth Avenue Hartsville, South Carolina 29550

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

UNIT IS SHUTDOWN FOR STEAM GENERATOR REPAIR.

♦ LAST IE SITE INSPECTION DATE: MARCH 31 - APRIL 3, 1980

♦ INSPECTION REPORT NO: 50-261/80-07

SUMMARY

INSPECTION SUMMARY

♦ THIS ROUTINE ANNOUNCED INSPECTION INVOLVED 28 INSPECTOR-HOURS ON SITE IN THE AREAS OF FOLLOWUP TO PREVIOUSLY IDENTIFIED TRAINING AND QUALITY ASSURANCE INSPECTION ITEMS AND PLANT TOUR. OF THE THREE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION MARCH 31 - APRIL 3 (80-07): THIS ROUTINE UNANNOUNCED INSPECTION INVOLVED 34 INSPECTOR-HOURS ON SITE IN THE AREAS OF OPERATIONS, LICENSEE EVENT REPORTS AND BULLETINS. OF THE THREE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
80-03/ 03L-0	02/21/80	03/21/80	30 DAY LER	PINHOLE LEAK CUT INTO C CVCS HOLDUP TANK INLET PIPING.
80-06/ 01L-P	03/24/80	03/24/80		FAILURE OF W MODEL NBFD-315 RELAYS DUE TO SHORTING OF COIL LEAD IN WIRES.

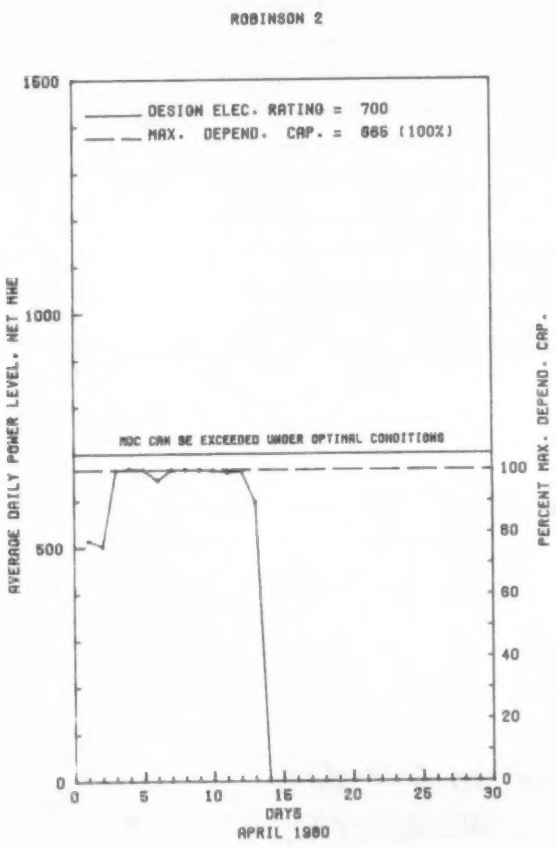
REVIEWED BY OMPA LOEB: T.C.Cintula DATE: 6·12·80

REVIEWED BY NRR DOR: Dan Neighbors DATE: 6/13/80

GPO 871-667

ROBINSON 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-261
2. Utility Contact: M. L. Watford (803) 383-4524
3. Licensed Thermal Power (MWt): 2300
4. Nameplate Rating (Gross MWe): 854 x 0.9 = 769
5. Design Electrical Rating (Net MWe): 700
6. Maximum Dependable Capacity (Gross MWe): 700
7. Maximum Dependable Capacity (Net MWe): 665
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): 2200 MWt = 665 MWe
10. Reasons for Restrictions, If Any:
EXCESSIVE MOISTURE CARRY-OVER TO H.P. TURBINE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	719.0	2,903.0	80,285.0
12. Hours Reactor Was Critical	311.7	2,178.5	62,385.4
13. Reactor Reserve Shutdown Hours	.3	157.5	876.8
14. Hours Generator On-Line	310.7	2,173.2	60,864.1
15. Unit Reserve Shutdown Hours	.0	.0	23.2
16. Gross Thermal Energy (MWH)	641,921	4,587,009	125,137,338
17. Gross Electrical Energy (MWH)	208,012	1,503,440	40,439,953
18. Net Electrical Energy (MWH)	194,498	1,424,517	38,329,619
19. Unit Service Factor	43.2	74.9	75.8
20. Unit Availability Factor	43.2	74.9	75.8
21. Unit Capacity Factor (MDC Net)	40.7	73.8	71.8
22. Unit Capacity Factor (DER Net)	38.6	70.1	68.2
23. Unit Forced Outage Rate	56.8	25.1	13.7

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
REFUELING/MAINTENANCE, JUNE, 6 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: 05/02/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
04-01	04/13/80	F	408.3	A	1	80-07	CB	PUMPXX	UNIT WAS SHUTDOWN DUE TO SEAL FAILURE IN "C" REACTOR COOLANT PUMP. DURING COOLING DOWN FOR RCP SEAL INSPECTION, A PRIMARY TO SECONDARY LEAK RATE OF .50 GPM WAS IDENTIFIED IN "B" STEAM GENERATOR. "C" RCP SEALS WERE INSPECTED AND THE NECESSARY MAINTENANCE WAS PERFORMED. 100% EDDY CURRENT TESTING IN ALL THREE STEAM GENERATORS RESULTED IN APPROXIMATELY 149 TUBES BEING PLUGGED.

SUMMARY

OPERATION WAS ROUTINE UNTIL 04/13/80, WHEN THE UNIT WAS SHUTDOWN TO REPAIR A SEAL IN THE "C" REACTOR COOLANT PUMP. EDDY CURRENT TESTING (100%) WAS PERFORMED IN ALL THREE STEAM GENERATORS FOLLOWING DISCOVERY OF AN EXCESSIVE PRIMARY-TO-SECONDARY LEAK RATE IN THE "B" STEAM GENERATOR.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

best copy available

SALEM 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	New Jersey/Salem 20 miles S of Wilmington, Del.
2.	Electric Reliability Council	Mid-Atlantic Area Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	3338
5.	Design Electrical Rating (MWE Net)	1090
6.	Date of Initial Criticality	December 11, 1976
7.	Date of Commercial Operation	June 30, 1977
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Delaware River

Utility & Contractor Information

10.	Licensee	Public Service Electric & Gas Co.
11.	Corporate Address	80 Park Place Newark, New Jersey 07101
12.	Corporate Contact	F. P. Librizzi General Manager - Electric Prod.
13.	Architect/Engineer	PSE&G
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	UE&C

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	IE Section Chief	R. R. Keimig
19.	IE Resident Inspector	L. J. Norrholm (609-935-3850) (FTS 341-3000)
20.	Licensing Project Manager	W. Ross
21.	Docket Number	50-272
22.	License Number/Date of Issue	DPR-70, 8/13/76
23.	Public Document Room	Salem Free Public Library 112 West Broadway Salem, New Jersey 08079

INSPECTION STATUS

ENFORCEMENT STATUS

FAILED TO ISOLATE THE INDIVIDUAL RESPONSIBLE FOR THE BADGE PASSOUT & LAST ACCESS CONTROL FUNCTION TO THE PROTECTED AREA. (7926 INFRACTION)

CONTRARY TO TS 3.0.4, PLANT MODE WAS CHANGED FROM 5 TO 2 WITH ONE CIRCUIT OF VITAL HEAT TRACE INOPERABLE. (8005 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

+ Tell-tale drain from spent fuel pool liner leak has decreased to approximately 78 milliliters/hr as a result of epoxy repairs. Evaluation is in progress to determine if leak is completely repaired and remaining flow is from condensate runoff.

PLANT STATUS:

Operating at full power.

+ LAST IE SITE INSPECTION DATE: 5/5/80

+ INSPECTION REPORT NO: 50-272/80-11

SUMMARY

INSPECTION SUMMARY

+ 50-272/80-02 - Jan 21-25: Unannounced physical security inspection (34 hours) by a region-based inspector of the approved physical security plan, security program audits, security organization, physical barriers (PA/VA), lighting, access control (search)(escorting)(vital areas), alarm station, detection aids, assessment aids, communications, testing and maintenance, compensatory measures, power supply, response, locks, keys and combinations and records and reports. Noncompliances: three (failure to forward a change to the Physical Security Plan to the NRC as required by 10 CFR 50.54(p); failure to assign watchmen only those responsibilities outlined in the plan; failure to revalidate vital area access control to all vital areas every 31 days).

+ 50-272/80-05 - Feb 3 - Mar 1: Routine inspections by the resident inspector (31 hours) of plant operations including: tours of the facility; log and record reviews; review of licensee events; core physics surveillance; and, followup on previous inspection items. Noncompliances: One (failure to meet system operability requirements prior to mode change as required by Technical Specification 3.0.4).

+ 50-272/80-07 - Mar 2-29: Routine inspections by the resident inspector (63 hours) of plant operations including tours of the facility; log and record reviews; review of licensee events; review of periodic reports; followup on implementation of small break LOCA guidelines; and, followup on previous inspection items. Noncompliances: None.

+ 50-272/80-08 - Mar 11-14: Routine, unannounced inspection (33 hours) of refueling startup testing including control rod drive operation and position indication, core power distribution, incore/excore detector calibration, reactor shutdown margin, isothermal temperature coefficient, power coefficient and control rod worth measurements. Noncompliances: None.

+ 50-272/80-09 - Feb 26: Enforcement conference held in NRC Region I between representatives of Public Service Electric and Gas Company and Region I staff to discuss: corrective actions relative to the simultaneous removal from service of all three emergency diesel generators on October 31, 1979; plant physical security matters; and radiation protection matters. Twelve hours of Region I staff time were expended for this meeting.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

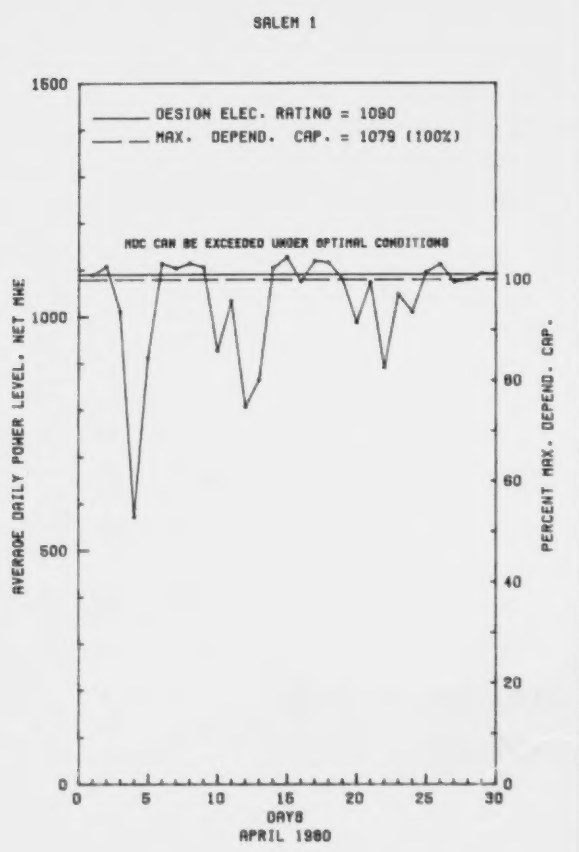
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
79-057/ 01T	09/07/79	09/20/79	Three seismic pipe hangers fail in service
79-059/ 03L	04/24/79	03/27/80	Loss of Residual heat removal flow in refueling mode due to electrical testing
80-003/ 03L	02/03/80	03/03/80	Failure of the air particulate detector due to a seized pump
80-010/ 03L	02/06/80	03/03/80	Failure to load test fuel handling manipulator crane in accordance with procedure criteria
80-011/ 03L	02/12/80	03/03/80	Failure of the containment air lock door to meet leak rate criteria of Technical Specifications
80-012/ 03L	02/05/80	03/03/80	Failure of the air particulate detector due to failure of the shear pins connecting the sampler drive unit
80-013/ 03L	02/27/80	03/27/80	Inoperability of one vital heat trace channel on RWST and associated flow path
80-014/ 03L	03/03/80	03/31/80	A ground developed on a power supply time that resulted in a loss of #12 station power transformer
80-015/ 03L	03/04/80	04/01/80	During normal operation the auxiliary annunciator typewriter failed due to a failed solid state component circuit board
80-016/ 03L	03/19/80	04/09/80	Due to personnel error the radiation monitoring system power supply was grounded which resulted in a voltage transient that tripped the air particulate detector breaker
80-017/ 01T	03/31/80	04/11/80	Potential inoperability of RCS wide range pressure transmitters during major LOCA with maximum containment flooding

REVIEWED BY OMPA LOEB: T.C. Cintula **DATE:** 6·12·80

REVIEWED BY NRR DOR: W.J. Ross **DATE:** 6/13/80

SALEM 1

AVERAGE DAILY POWER LEVEL(MWe)



OPERATING STATUS

1. Docket Number: 50-272
2. Utility Contact: L. K. Miller (609) 365-7000
3. Licensed Thermal Power (MWt): 3338
4. Nameplate Rating (Gross MWe): 1300 x 0.9 = 1170
5. Design Electrical Rating (Net MWe): 1090
6. Maximum Dependable Capacity (Gross MWe): 1124
7. Maximum Dependable Capacity (Net MWe): 1079
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	719.0	2,903.0	24,864.0
12. Hours Reactor Was Critical	719.0	2,812.5	12,900.8
13. Reactor Reserve Shutdown Hours	.0	.0	22.7
14. Hours Generator On-Line	719.0	2,713.3	12,244.4
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	2,269,934	8,245,382	35,781,678
17. Gross Electrical Energy (MWH)	774,940	2,787,540	11,955,040
18. Net Electrical Energy (MWH)	744,437	2,667,730	11,297,320
19. Unit Service Factor	100.0	93.5	49.2
20. Unit Availability Factor	100.0	93.5	49.2
21. Unit Capacity Factor (MDC Net)	96.0	85.2	42.1
22. Unit Capacity Factor (DER Net)	95.0	84.3	41.7
23. Unit Forced Outage Rate	.0	4.5	39.3

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUELING 09/20/80 - 12/07/80/
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-091	04/04/80	F	.0	B	5		HH	PUMPXX	INSPECTION OF #12 SGFP INBOARD JOURNAL BEARING.
80-093	04/10/80	F	.0	A	5		HH	PUMPXX	REPAIR #13 CONDENSATE PUMP BEARING.
80-095	04/12/80	F	.0	A	5		HH	PUMPXX	OIL LEAK ON #12 SGFP OUTBOARD BEARING.
80-097	04/17/80	F	.0	A	5		HF	FILTER	REPAIR #11A AND #12A TRAVELING SCREENS.
80-099	04/20/80	F	.0	A	5		HH	FILTER	CLEAN #11, 12, 13 CONDENSATE PUMP SUCTION STRAINERS.
80-100	04/21/80	F	.0	A	5		HH	FILTER	CLEAN #11 HEATER DRAIN SUCTION STRAINER.
80-102	04/22/80	F	.0	A	5		HH	FILTER	CLEAN #11 CONDENSATE PUMP SUCTION STRAINER.
80-104	04/22/80	F	.0	A	5		HH	FILTER	CLEAN #11 CONDENSATE PUMP SUCTION STRAINER.
80-106	04/26/80	F	.0	A	5		HH	FILTER	REPAIR #11B TRAVELING SCREEN.

SUMMARY

OPERATION WAS ROUTINE DURING THE REPORT PERIOD, WITH SEVERAL LOAD REDUCTIONS TO CLEAN CONDENSATE AND HEATER DRAIN SUCTION STRAINERS AND TO REPAIR TRAVELING SCREENS. REPAIRS WERE ALSO MADE TO A STEAM GENERATOR FEED PUMP INBOARD JOURNAL BEARING AND OUTBOARD BEARING OIL SEAL.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintanance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

SAN ONOFRE 1

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	California/San Diego 5 miles S of San Clemente
2.	Electric Reliability Council	Western Systems Coordinating Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	1347
5.	Design Electrical Rating (MWE Net)	436
6.	Date of Initial Criticality	June 14, 1967
7.	Date of Commercial Operation	January 1, 1968
8.	Condenser Cooling Method	Circulating sea water
9.	Condenser Cooling Water Source	Pacific Ocean

Utility & Contractor Information

10.	Licensee	Southern California Edison Co.
11.	Corporate Address	2244 Walnut Grove Avenue Rosemead, California 91770
12.	Corporate Contact	R. E. Woodbury V.P. & General Counsel
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	V
17.	IE Branch Chief	J. L. Crews
18.	IE Section Chief	B. H. Faulkenberry
19.	IE Resident Inspector	L. F. MILLER (714-492-2641)
20.	Licensing Project Manager	T. Wambach
21.	Docket Number	50-206
22.	License Number/Date of Issue	DPR-13, 3/27/67
23.	Public Document Room	Mission Viejo Branch Library 24851 Chrisanta Drive Mission Viejo, California 92676

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

SYSTEMS AND COMPONENT PROBLEMS:

Refueling outage commenced two days earlier than scheduled due to increase in primary to secondary (S/G tube) leakage (PNO-V-80-24).

FACILITY ITEMS (PLANS AND PROCEDURES):

Refueling.

MANAGERIAL ITEMS

None.

PLANT STATUS:

Plant shut down for refueling outage.

LAST IE SITE INSPECTION DATE: 04/01-30/80+

INSPECTION REPORT NO: 50-206/80-12+

SUMMARY

INSPECTION SUMMARY

+ Inspection on March 10-13, 1980 (Report No. 50-206/80-05) Areas Inspected: Routine, unannounced inspection of document control and records programs and independent effort. This inspection involved 23 inspector hours onsite by one NRC inspector.

Results: No items of noncompliance or deviations were identified.

+ Inspection on March 1 - April 4, 1980 (Report No. 50-206/80-09) Areas Inspected: Routine resident inspection of plant operations and followup of licensee event. The inspection involved 94 inspector hours onsite by seven NRC inspectors and management personnel.

Results: Of the two areas examined, two items of noncompliance were identified in the followup of the March 10, 1980 licensee event concerning loss of the salt water cooling system (Infraction - Failure to follow emergency procedure) and (Infraction - Operation outside of a limiting condition for operation).

+ Inspection on March 1 - April 4, 1980 (Report No. 50-206/80-10) Report being prepared; to be reported next month.

+ Inspection on April 21, 1980 (Report No. 50-206/80-11) Report being prepared; to be reported next month.

+ Inspection on April 1-30, 1980 (Report No. 50-206/80-12) Report being prepared; to be reported next month.

+ Inspection on April 23-25, 1980 (Report No. 50-206/80-13) Report being prepared; to be reported next month.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

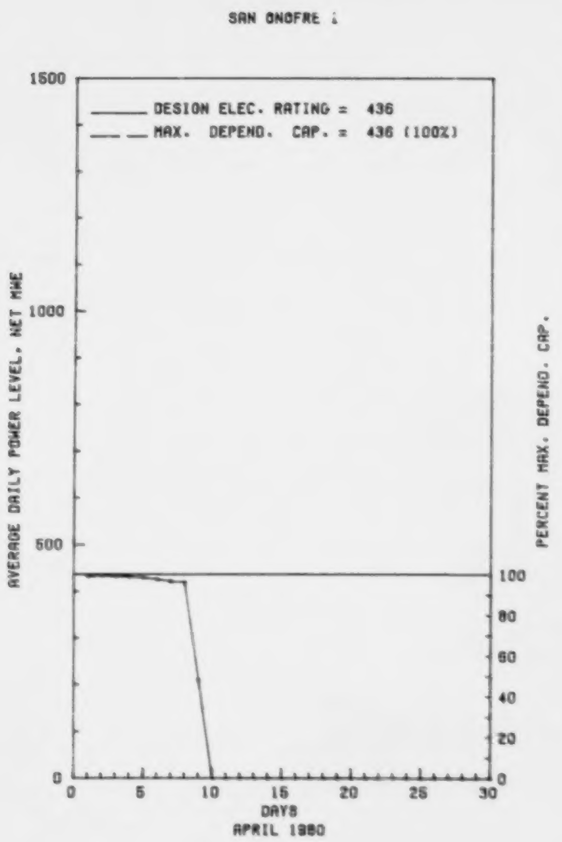
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-07/ 03L-0	02-24-80	03-20-80	North charging pump miniflow line leak - Early repair not to standard.
80-08/ 03L-0	03-18-80	04-04-80	Salt water cooling pmp pipe support loose and corroded anchors.
80-09/ 04L-0	03-31-80	04-02-80	Only six of 11 required Benthic samples taken during first quarter of 1980.
80-10/ 03L-0	03-18-80	04-09-80	High temperature on south chgng pump thrust bearing required pump to be repaired.
80-11/ 43X-0	03-06-80	04-08-80	Overfilling of prt to vent tank resulted in rupturing the relief disk.
80-12/ 03L-0	03-24-80	04-20-80	No. 2 D/G fuel oil transfer pumps tripped on overload; D/G inoperable.
80-13/ 03L-0	03-29-80	04-28-80	Control room normal and emergency air intake dampers failed to actuate.
80-15/ 01T-0	04-22-80	05-05-80	While in cold shutdown, all AC power was lost for four minutes.

REVIEWED BY OMPA LOEB: T. C. Cintula DATE: 6·12·80

REVIEWED BY NRR DOR: J. V. Wambach DATE: 6/16/80

SAN ONOFRE 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-206
2. Utility Contact: R. G. Moran (714) 492-7700
3. Licensed Thermal Power (MWt): 1347
4. Nameplate Rating (Gross MWe): 500 x 0.9 = 450
5. Design Electrical Rating (Net MWe): 436
6. Maximum Dependable Capacity (Gross MWe): 456
7. Maximum Dependable Capacity (Net MWe): 436
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	719.0	2,903.0	112,879.0
12. Hours Reactor Was Critical	206.5	1,979.1	84,611.3
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	206.3	1,961.6	81,114.0
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	270,210	2,552,787	104,084,261
17. Gross Electrical Energy (MWH)	90,600	855,000	35,526,434
18. Net Electrical Energy (MWH)	85,933	816,676	33,664,042
19. Unit Service Factor	28.7	67.6	71.9
20. Unit Availability Factor	28.7	67.6	71.9
21. Unit Capacity Factor (MDC Net)	27.4	64.5	68.4
22. Unit Capacity Factor (DER Net)	27.4	64.5	68.4
23. Unit Forced Outage Rate	.0	2.8	9.2

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 06/01/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
55	04/09/80	S	512.7	C	1		ZZ	ZZZZZZ	REFUELING OUTAGE.

SUMMARY

THE UNIT WAS SHUTDOWN ON 04/09/80 FOR REFUELING.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

ST. LUCIE 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Florida/St. Lucie 12 miles SE of Ft. Pierce
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2560
5.	Design Electrical Rating (MWE Net)	802
6.	Date of Initial Criticality	April 22, 1976
7.	Date of Commercial Operation	December 21, 1976
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Atlantic Ocean

Utility & Contractor Information

10.	Licensee	Florida Power & Light Co.
11.	Corporate Address	9250 W. Flagler Street P. O. Box 529100 Miami, Florida 33152
12.	Corporate Contact	R. E. Uhrig, V.P.
13.	Architect/Engineer	Ebasco
14.	Nuclear Steam Supply System	Combustion Engineering
15.	Constructor	Ebasco

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. LEWIS
18.	IE Section Chief	D. R. QUICK
19.	IE Resident Inspector	S. A. ELROD
20.	Licensing Project Manager	C. Nelson
21.	Docket Number	50-335
22.	License Number/Date of Issue	DPR-67, March 1, 1976
23.	Public Document Room	Indian River Junior College Library 3209 Virginia Avenue Ft. Pierce, Florida 33450

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO TECHNICAL SPECIFICATION 3.9.2, ON APRIL 1, 1980 DURING REFUELING OPERATIONS, FLUX MONITOR INDICATION WAS NOT AUDIBLE AT THE REFUEL CAVITY WHERE CORE ALTERATIONS WERE BEING MADE. (8008 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

UNIT IS IN A REFUELING OUTAGE SINCE MARCH 15, 1980.

LAST IE SITE INSPECTION DATE: APRIL 29 - MAY 2, 1980

INSPECTION REPORT NO: 50-335/80-13

SUMMARY

INSPECTION SUMMARY

INSPECTION FEBRUARY 25-26 (80-01): THIS ROUTINE, UANNOUNCED INSPECTION INVOLVED 50 INSPECTOR-HOURS ON SITE IN THE AREAS OF QUALITY CONTROL AND CONFIRMATORY MEASUREMENTS INCLUDING REVIEW OF CHEMICAL AND RADIOCHEMICAL PROCEDURES; AND COMPARISON OF RADIOCHEMICAL ANALYSES OF SPLIT SAMPLES BY THE LICENSEE AND THE NRC REGION II MOBILE LABORATORY. OF THE THREE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION JANUARY 28 - FEBRUARY 1 (80-04): INCLUDED REVIEW OF THE SECURITY PLAN, SECURITY PROGRAM AUDIT, SECURITY ORGANIZATION, PHYSICAL BARRIERS (PROTECTED AREA), PHYSICAL BARRIERS (VITAL AREA), LIGHTING, ACCESS CONTROL (ID AUTHORIZATION AND BADGING), ACCESS CONTROL (SEARCH), ACCESS CONTROL (ESCORTING), ACCESS CONTROL (VITAL AREA), ALARM STATIONS, DETECTION AIDS, ASSESSMENT AIDS, COMMUNICATIONS, AND LOCKS AND KEYS. THE INSPECTION INVOLVED 75 INSPECTOR-HOURS ON SITE BY 2 NRC INSPECTORS. THE INSPECTION WAS BEGUN DURING REGULAR HOURS; 19 INSPECTOR-HOURS WERE ACCOMPLISHED DURING OFFSHIFT PERIODS. THE LICENSEE WAS FOUND TO BE IN COMPLIANCE WITH NRC REQUIREMENTS WITHIN THE 15 AREAS EXAMINED DURING THE INSPECTION EXCEPT FOR THE FOLLOWING 4 ITEMS: (1) SECURITY ORGANIZATION - FAILURE TO ESTABLISH, MAINTAIN AND FOLLOW ADEQUATE WRITTEN PROCEDURES (INFRACTION); (2) PHYSICAL BARRIERS (VITAL AREA) - FAILURE TO MAINTAIN A VITAL AREA BARRIER AS COMMITTED TO IN THE SECURITY PLAN (INFRACTION); (3) ACCESS CONTROL (SEARCH) - FAILURE TO PROPERLY SEARCH PERSONNEL (INFRACTION); (4) ACCESS CONTROL (VITAL AREA) - FAILURE TO CONTROL ALL POINTS OF PERSONNEL ACCESS INTO VITAL AREAS (INFRACTION); (5) SECURITY ORGANIZATION - COMMITMENTS TO TRAIN AND QUALIFY GUARDS TO MEET OR EXCEED THE REQUIREMENTS OF REGULATORY GUIDE 5.20; AND (6) PHYSICAL BARRIERS (VITAL AREA) - DESIGN OF BARRIERS AND LOCKING HARDWARE.

INSPECTION MARCH 24-27 (80-05): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 44 INSPECTOR-HOURS ON SITE AND AT THE CORPORATE OFFICE IN THE AREA OF TECHNICAL SPECIFICATION SURVEILLANCE REQUIREMENTS, ATTENDANCE AT THE SHORT TERM LESSONS LEARNED REVIEW CONDUCTED BY THE DIVISION OF OPERATING REACTORS, AND ATTENDANCE AT A REVIEW OF 10 CFR 50 PART 21 REPORTING REQUIREMENTS. OF THE THREE AREAS INSPECTED, NO ITEM OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION APRIL 8-10 (80-09): THIS SPECIAL, ANNOUNCED INSPECTION INVOLVED 16 INSPECTOR-HOURS ON SITE IN THE AREAS OF INSTALLED EQUIPMENT REVIEW WITH RESPECT TO IE BULLETIN 79-01B. OF THE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION APRIL 1-3 (80-08): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 24 INSPECTOR-HOURS ON SITE IN THE AREAS OF PREPARATION FOR REFUELING AND REFUELING ACTIVITIES IN PREPARATION FOR FUEL CYCLE FOUR. OF THE TWO AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN ONE AREA; ONE ITEM OF NONCOMPLIANCE WAS FOUND IN THE OTHER AREA (INFRACTION - FAILURE TO HAVE A FLUX MONITOR INDICATION AUDIBLE IN CONTAINMENT DURING CORE ALTERATIONS).

AN ENFORCEMENT CONFERENCE WAS CONDUCTED WITH LICENSEE MANAGEMENT ON APRIL 14 (80-11), TO DISCUSS THE FINDINGS OF THE SECURITY INSPECTION JANUARY 28 - FEBRUARY 1, 1980 AND TO REVIEW PREVIOUS ENFORCEMENT HISTORY.

REPORTS RECEIVED FROM LICENSEE

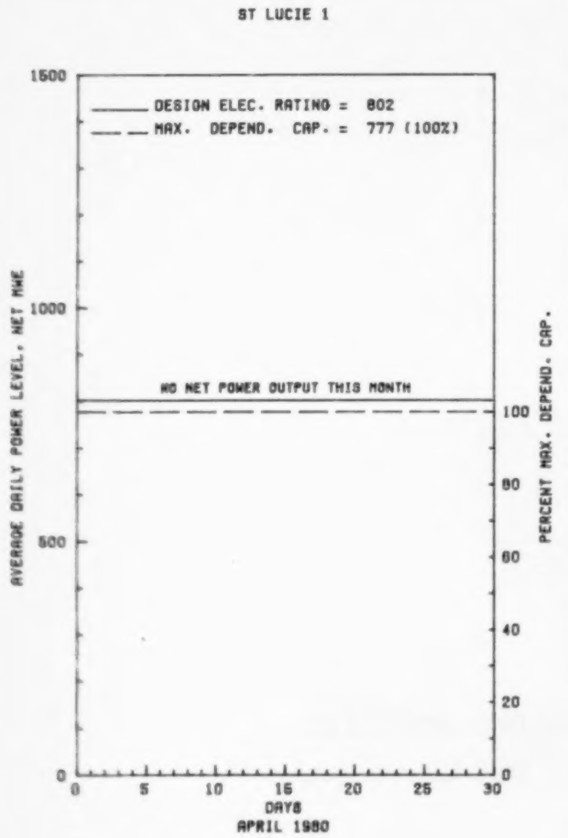
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
80-01/ 03L-0	02/13/80	03/12/80	30 DAY LER	ZONE OF MIXING-SURFACE TEMP RISE.
80-09/ 03L-0	02/08/80	03/07/80	30 DAY LER	REACTOR COOLANT SYSTEM UNIDENTIFIED LEAKAGE HIGH DUE TO VOLUME CONTROL TANK RELIEF VALVE CYCLING OPEN AND SHUT.
80-10/ 03L-0	02/17/80	03/17/80	30 DAY LER	DROPPED CEA.
80-11/ 03L-0	02/19/80	03/19/80	30 DAY LER	SETPOINT DRIFT-REFUELING WATER TANK LEVEL.
80-13/ 09T	03/17/80	03/25/80		DIESEL GENERATOR FIELD WIRING ERROR.

REVIEWED BY OMPA LOEB: T.C. Cintula **DATE:** 6.12.80

REVIEWED BY NRR DOR: C Nelson **DATE:** 6-13-80

ST. LUCIE 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-335
2. Utility Contact: V. T. Chilson (305) 552-3824
3. Licensed Thermal Power (MWt): 2560
4. Nameplate Rating (Gross MWe): 1000 x 0.85 = 850
5. Design Electrical Rating (Net MWe): 802
6. Maximum Dependable Capacity (Gross MWe): 822
7. Maximum Dependable Capacity (Net MWe): 777
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	719.0	2,903.0	29,447.0
12. Hours Reactor Was Critical	.0	1,783.1	23,293.0
13. Reactor Reserve Shutdown Hours	.0	.0	129.4
14. Hours Generator On-Line	.0	1,780.6	22,618.7
15. Unit Reserve Shutdown Hours	.0	.0	32.0
16. Gross Thermal Energy (MWH)	0	4,487,283	54,756,849
17. Gross Electrical Energy (MWH)	0	1,449,650	17,744,150
18. Net Electrical Energy (MWH)	-2,703	1,366,133	16,694,829
19. Unit Service Factor	.0	61.3	76.8
20. Unit Availability Factor	.0	61.3	76.9
21. Unit Capacity Factor (MDC Net)	.0	60.6	73.0
22. Unit Capacity Factor (DER Net)	.0	58.7	70.7
23. Unit Forced Outage Rate	.0	.9	5.1

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 05/10/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
4	03/15/80	S	719.0	C	4		RC	FUELXX	SCHEDULED REFUELING, MAINTENANCE, INSPECTIONS AND OUTAGE CONTINUES.

SUMMARY

THE UNIT REMAINED SHUTDOWN THROUGHOUT THE REPORT PERIOD FOR A REFUELING, MAINTENANCE, AND INSPECTION OUTAGE.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

SURRY 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Virginia/Surry 17 miles NW of Newport News
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2441
5.	Design Electrical Rating (MWE Net)	822
6.	Date of Initial Criticality	July 1, 1972
7.	Date of Commercial Operation	December 22, 1972
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	James River

Utility & Contractor Information

10.	Licensee	Virginia Electric and Power Company
11.	Corporate Address	P. O. Box 26666 Richmond, Virginia 23261
12.	Corporate Contact	J. H. Ferguson Exec. V.P. - Power
13.	Architect/Engineer	Stone & Webster
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R.C. Lewis
18.	IE Section Chief	P. J. Kellogg
19.	IE Resident Inspector	D. Burke (804) 357-2101
20.	Licensing Project Manager	D. Neighbors
21.	Docket Number	50-280
22.	License Number/Date of Issue	DPR-32, 5/25/72
23.	Public Document Room	Swem Library College of William and Mary Williamsburg, Virginia 23185

INSPECTION STATUS

ENFORCEMENT STATUS

AS REQUIRED BY 10 CFR 50, APPENDIX E, ITEM IV.H AND SECTION 7.1.1 OF THE SURRY NUCLEAR POWER STATION EMERGENCY PLAN, PERIODIC TRAINING SHALL BE PROVIDED TO ALL FACILITY PERSONNEL. MOREOVER, TECHNICAL SPECIFICATION 6.4.D REQUIRES IN PART THAT EMERGENCY PROCEDURES SHALL BE MAINTAINED AND FOLLOWED. EMERGENCY PLAN IMPLEMENTATION PROCEDURE 21 REQUIRES, IN PART, ANNUAL RETRAINING FOR ALL FACILITY PERSONNEL. FOUR OF FIVE INDIVIDUALS DESIGNATED AS EMERGENCY DIRECTORS HAD NOT RECEIVED ANNUAL REFRESHER TRAINING DURING THE PERIOD AUGUST 8, 1978 TO MARCH 11, 1980.
(8004 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

RE-ANALYSIS AND REVIEW OF SEISMIC DESIGN DEFICIENCIES ARE CONTINUING.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

UNIT IS CURRENTLY SHUTDOWN FOR MAIN TURBINE INSPECTION AND CORRECTION OF SEISMIC DESIGN ERRORS.

♦ LAST IE SITE INSPECTION DATE: MAY 2, 1980

♦ INSPECTION REPORT NO: 50-280/80-19

SUMMARY

INSPECTION SUMMARY

♦ INSPECTION MARCH 10-14 (80-04): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 30 INSPECTOR-HOURS ON SITE IN THE AREAS OF COORDINATION WITH OUTSIDE SUPPORT AGENCIES; EMERGENCY FACILITIES, EQUIPMENT AND PROCEDURES; EMERGENCY ORGANIZATION; EMERGENCY TRAINING; EMERGENCY PLANNING AUDITS; EMERGENCY DRILLS; FOLLOWUP ON PREVIOUSLY IDENTIFIED INSPECTION FINDINGS AND FOLLOWUP ON IE BULLETINS. OF THE EIGHT AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN SEVEN AREAS; ONE ITEM OF NONCOMPLIANCE WAS FOUND IN ONE AREA (INFRACTION - FAILURE TO CONDUCT EMERGENCY RETRAINING FOR DESIGNATED EMERGENCY DIRECTORS).

INSPECTION MARCH 4-7 (80-06): THIS SPECIAL, ANNOUNCED INSPECTION INVOLVED 24 INSPECTOR-HOURS ON SITE IN THE AREA OF SMALL BREAK LOCA, EMERGENCY PROCEDURES, TRAINING AND OPERATOR KNOWLEDGE. WITHIN THE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION APRIL 15-18 (80-08): THIS SPECIAL, UNANNOUNCED INSPECTION INVOLVED 15 INSPECTOR-HOURS ON SITE IN THE AREAS OF FIRE PROTECTION/PREVENTION. OF THE AREAS INSPECTED, NO ITEMS OF DEVIATIONS WERE IDENTIFIED. HOWEVER, TWO ITEMS OF NONCOMPLIANCE WERE FOUND (INFRACTION - NONFUNCTIONAL FIRE DOOR BETWEEN UNIT 2 EMERGENCY SWITCHGEAR ROOM AND UNIT 2 SERVICE BUILDING CABLE VAULT; AND, INFRACTION - FAILURE TO IMPLEMENT FIRE PROTECTION PROCEDURES FOR SPECIAL PROCESSES INVOLVING IGNITION SOURCES).

INSPECTION APRIL 7-9 (80-09): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 16 INSPECTOR-HOURS ON SITE IN THE AREAS OF CONSTRUCTION STATUS, LICENSEE ACTION ON PREVIOUS INSPECTION FINDINGS, AND REVIEW OF QUALITY RECORDS. OF THE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION FEBRUARY 27 - MARCH 4 AND MARCH 10 - APRIL 4 (80-13): THIS ROUTINE INSPECTION BY THE RESIDENT INSPECTOR INVOLVED 65 INSPECTOR-HOURS ON SITE IN THE AREAS OF PLANT OPERATIONS AND OPERATING RECORDS, PLANT MODIFICATION AND MAINTENANCE, FOLLOWUP OF IE BULLETINS, PLANT SECURITY, AND PREVIOUSLY IDENTIFIED ITEMS. OF THE FIVE AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE FOUND IN THREE AREAS; ONE APPARENT DEVIATION WAS FOUND IN THE PLANT MODIFICATION AREA (THE FACILITY NOT BUILT AS DESCRIBED IN FSAR) AND ONE APPARENT ITEM OF NONCOMPLIANCE WAS FOUND IN THE AREA OF PLANT SECURITY (2.790).

INSPECTION APRIL 15-18 (80-15): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 15 INSPECTOR-HOURS ON SITE IN THE AREAS OF RECORDS PROGRAM IMPLEMENTATION AND TECHNICAL WORKING SESSION WITH LICENSEE AND NRR CONCERNING THE INSERVICE TESTING, PUMP AND VALVE PROGRAM. OF THE TWO AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

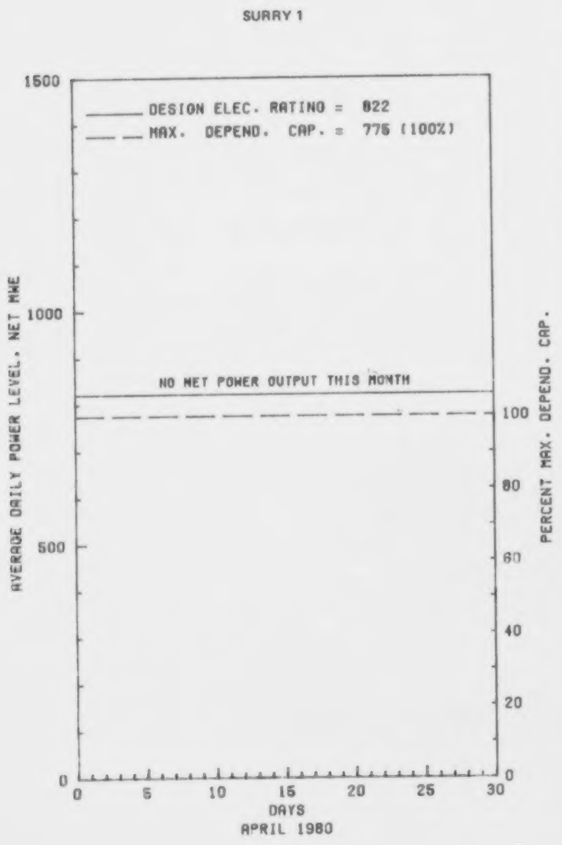
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
79-34 /01T-0	12/07/79	12/20/79	14 DAY LER	NONCONSERVATISM IN DESIGN OF CONTROL CIRCUITRY FOR THE CONTAINMENT ISOLATION VALVES FROM THE MAIN AIR EJECTOR DISCHARGE.
80-15/ 03L-0	02/18/80	03/14/80	30 DAY LER	COMPONENT COOLING WATER RADIATION ALARM SETPOINT DRIFTED HIGH.
80-16/ 03L-0	02/18/80	03/14/80	30 DAY LER	LIQUID WASTE TANK 11B PARTIALLY RELEASED WITHOUT SAMPLING.
80-17/ 03L-0	02/18/80	03/14/80	30 DAY LER	VACUUM PUMP PROCESS MONITORS FOUND OUT OF SERVICE.
80-19/ 03L-0	03/04/80	03/28/80	30 DAY LER	LIQUID WASTE TANKS 7A AND 7B VOLUMES FOUND TO BE INCORRECT.
80-20/ 01T-0	03/10/80	03/24/80	14 DAY LER	REDUNDANT HEAT TRACE CIRCUITS TO THE BORIC ACID FILTER SYSTEMS TAGGED OUT.

REVIEWED BY OMPA LOEB: T. C. Cintula **DATE:** 6·12·80

REVIEWED BY NRR DOR: D. Neighbors **DATE:** 6/13/80

SURRY 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-280
2. Utility Contact: O. J. Costello (804) 357-3184
3. Licensed Thermal Power (MWt): 2441
4. Nameplate Rating (Gross MWe): 942 x 0.9 = 848
5. Design Electrical Rating (Net MWe): 822
6. Maximum Dependable Capacity (Gross MWe): 811
7. Maximum Dependable Capacity (Net MWe): 775
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	719.0	2,903.0	64,487.0
12. Hours Reactor Was Critical	0	1,016.2	39,767.8
13. Reactor Reserve Shutdown Hours	.0	182.6	3,731.5
14. Hours Generator On-Line	.0	1,010.4	38,915.4
15. Unit Reserve Shutdown Hours	.0	183.6	3,736.2
16. Gross Thermal Energy (MWH)	0	2,438,340	90,153,212
17. Gross Electrical Energy (MWH)	0	785,655	29,466,153
18. Net Electrical Energy (MWH)	0	746,810	27,973,709
19. Unit Service Factor	.0	34.8	60.3
20. Unit Availability Factor	.0	41.1	66.1
21. Unit Capacity Factor (MDC Net)	.0	33.2	56.0
22. Unit Capacity Factor (DER Net)	.0	31.3	52.8
23. Unit Forced Outage Rate	100.0	65.2	27.0

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUELING - SEPTEMBER, - 6 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: 05/09/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-4	02/19/80	F	719.0	D	4	80-018			CONTINUATION OF SHUTDOWN FOR PIPING RESTRAINT MODIFICATION AND TURBINE ROTOR INSPECTION WHICH BEGAN 02/19/80.

SUMMARY

THE UNIT REMAINED SHUTDOWN THROUGHOUT THE REPORT PERIOD FOR PIPING RESTRAINT MODIFICATIONS AND TURBINE REPAIRS.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

SURRY 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Virginia/Surry 17 miles NW of Newport News
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2441
5.	Design Electrical Rating (MWE Net)	822
6.	Date of Initial Criticality	March 7, 1973
7.	Date of Commercial Operation	May 1, 1973
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	James River

Utility & Contractor Information

10.	Licensee	Virginia Electric and Power Company
11.	Corporate Address	P. O. Box 26666 Richmond, Virginia 23261
12.	Corporate Contact	J. H. Ferguson Exec. V.P. - Power
13.	Architect/Engineer	Stone & Webster
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R.C. Lewis
18.	IE Section Chief	P. J. Kellog
19.	IE Resident Inspector	D. Burke (804) 357-2101
20.	Licensing Project Manager	D. Neighbors
21.	Docket Number	50-281
22.	License Number/Date of Issue	DPR-37, 1/29/73
23.	Public Document Room	Swem Library College of William and Mary Williamsburg, Virginia 23185

INSPECTION STATUS

ENFORCEMENT STATUS

AS REQUIRED BY 10 CFR 50, APPENDIX E, ITEM IV.H AND SECTION 7.1.1 OF THE SURRY NUCLEAR POWER STATION EMERGENCY PLAN, PERIODIC TRAINING SHALL BE PROVIDED TO ALL FACILITY PERSONNEL. MOREOVER, TECHNICAL SPECIFICATION 6.4.D REQUIRES IN PART THAT EMERGENCY PROCEDURES SHALL BE MAINTAINED AND FOLLOWED. EMERGENCY PLAN IMPLEMENTATION PROCEDURE 21 REQUIRES, IN PART, ANNUAL RETRAINING FOR ALL FACILITY PERSONNEL. FOUR OF FIVE INDIVIDUALS DESIGNATED AS EMERGENCY DIRECTORS HAD NOT RECEIVED ANNUAL REFRESHER TRAINING DURING THE PERIOD AUGUST 8, 1978 TO MARCH 11, 1980.
(8006 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

SYSTEM MODIFICATIONS FOR SEISMIC SUPPORT REANALYSIS AND NUREG 0578 ARE CONTINUING.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

UNIT IS CURRENTLY SHUT DOWN AND DEFUELED. THE STEAM GENERATOR REPLACEMENT PROJECT COMPLETED.

+ LAST IE SITE INSPECTION DATE: MAY 2, 1980

+ INSPECTION REPORT NO: 50-281/80-19

SUMMARY

INSPECTION SUMMARY

+ INSPECTION MARCH 10-14 (80-06): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 30 INSPECTOR-HOURS ON SITE IN THE AREAS OF COORDINATION WITH OUTSIDE SUPPORT AGENCIES; EMERGENCY FACILITIES, EQUIPMENT AND PROCEDURES; EMERGENCY ORGANIZATION; EMERGENCY TRAINING; EMERGENCY PLANNING AUDITS; EMERGENCY DRILLS; FOLLOWUP ON PREVIOUSLY IDENTIFIED INSPECTION FINDINGS AND FOLLOWUP ON IE BULLETINS. OF THE EIGHT AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN SEVEN AREAS; ONE ITEM OF NONCOMPLIANCE WAS FOUND IN ONE AREA (INFRACTION - FAILURE TO CONDUCT EMERGENCY RETRAINING FOR DESIGNATED EMERGENCY DIRECTORS).

INSPECTION MARCH 4-7 (80-08): THIS SPECIAL, ANNOUNCED INSPECTION INVOLVED 24 INSPECTOR-HOURS ON SITE IN THE AREA OF SMALL BREAK LOCA, EMERGENCY PROCEDURES, TRAINING AND OPERATOR KNOWLEDGE. WITHIN THE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION APRIL 15-18 (80-09): THIS SPECIAL, UNANNOUNCED INSPECTION INVOLVED 15 INSPECTOR-HOURS ON SITE IN THE AREAS OF FIRE PROTECTION/PREVENTION. OF THE AREAS INSPECTED, NO ITEMS OF DEVIATIONS WERE IDENTIFIED. HOWEVER, TWO ITEMS OF NONCOMPLIANCE WERE FOUND (INFRACTION - NONFUNCTIONAL FIRE DOOR BETWEEN UNIT 2 EMERGENCY SWITCHGEAR ROOM AND UNIT 2 SERVICE BUILDING CABLE VAULT; AND, INFRACTION - FAILURE TO IMPLEMENT FIRE PROTECTION PROCEDURES FOR SPECIAL PROCESSES INVOLVING IGNITION SOURCES).

INSPECTION APRIL 7-9 (80-10): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 16 INSPECTOR-HOURS ON SITE IN THE AREAS OF CONSTRUCTION STATUS, LICENSEE ACTION ON PREVIOUS INSPECTION FINDINGS, AND REVIEW OF QUALITY RECORDS. OF THE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION FEBRUARY 27 - MARCH 4 AND MARCH 10 - APRIL 4 (80-14): THIS ROUTINE INSPECTION BY THE RESIDENT INSPECTOR INVOLVED 65 INSPECTOR-HOURS ON SITE IN THE AREAS OF PLANT OPERATIONS AND OPERATING RECORDS, PLANT MODIFICATION AND MAINTENANCE, FOLLOWUP OF IE BULLETINS, PLANT SECURITY, AND PREVIOUSLY IDENTIFIED ITEMS. OF THE FIVE AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE FOUND IN THREE AREAS; ONE APPARENT DEVIATION WAS FOUND IN THE PLANT MODIFICATION AREA (THE FACILITY NOT BUILT AS DESCRIBED IN FSAR) AND ONE APPARENT ITEM OF NONCOMPLIANCE WAS FOUND IN THE AREA OF PLANT SECURITY (2.790).

INSPECTION APRIL 15-18 (80-16): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 15 INSPECTOR-HOURS ON SITE IN THE AREAS OF RECORDS PROGRAM IMPLEMENTATION AND TECHNICAL WORKING SESSION WITH LICENSEE AND NRR CONCERNING THE INSERVICE TESTING, PUMP AND VALVE PROGRAM. OF THE TWO AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

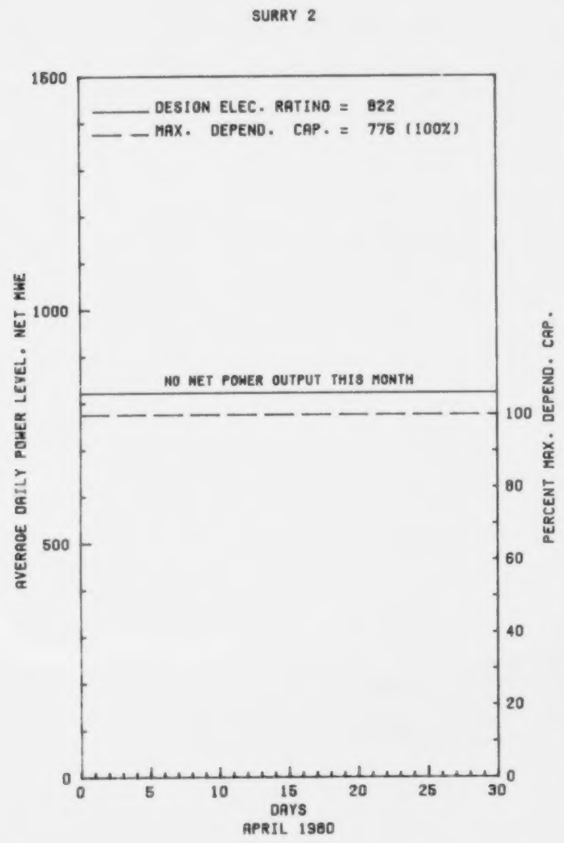
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
80-01/ 01T-0	03/10/80	03/24/80	14 DAY LER	NUCLEAR INSTRUMENTATION CALIBRATION PROCEDURE ERROR RESULTED IN EXCEEDING AXIAL FLUX DIFFERENCE LIMITS.

REVIEWED BY OMPA LOEB: T.C. CINTULA **DATE:** 6.12.80

REVIEWED BY NRR DOR: **DATE:** 6/13/80

SURRY 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-281
2. Utility Contact: B. J. Costello (804) 357-3184
3. Licensed Thermal Power (MWt): 2441
4. Nameplate Rating (Gross MWe): 942 x 0.9 = 848
5. Design Electrical Rating (Net MWe): 822
6. Maximum Dependable Capacity (Gross MWe): 811
7. Maximum Dependable Capacity (Net MWe): 775
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	719.0	2,903.0	61,367.0
12. Hours Reactor Was Critical	.0	.0	34,499.9
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	.0	.0	33,996.2
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	0	79,194,083
17. Gross Electrical Energy (MWH)	0	0	25,868,844
18. Net Electrical Energy (MWH)	0	0	24,536,605
19. Unit Service Factor	.0	.0	55.4
20. Unit Availability Factor	.0	.0	55.4
21. Unit Capacity Factor (MDC Net)	.0	.0	51.6
22. Unit Capacity Factor (DER Net)	.0	.0	48.6
23. Unit Forced Outage Rate	.0	.0	21.0

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 06/09/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-2	02/04/79	S	719.0	D	4				MODIFICATIONS BEING MADE AS A RESULT OF SHOW CAUSE ORDER FOR SEISMIC ANALYSIS AND IE BULLETIN 79-14.

SUMMARY

THE UNIT REMAINED SHUTDOWN THROUGHOUT THE REPORT PERIOD FOR PIPING RESTRAINT MODIFICATIONS AND REFUELING.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

THREE MILE ISLAND 1

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Pennsylvania/Dauphin 10 miles SE of Harrisburg
2.	Electric Reliability Council	Mid-Atlantic Area Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2535
5.	Design Electrical Rating (MWE Net)	819
6.	Date of Initial Criticality	June 5, 1974
7.	Date of Commercial Operation	September 2, 1974
8.	Condenser Cooling Method	Closed
9.	Condenser Cooling Water Source	Susquehanna River

Utility & Contractor Information

10.	Licensee	Metropolitan Edison Company
11.	Corporate Address	P. O. Box 542 Reading, Pennsylvania
12.	Corporate Contact	R. C. Arnold, Sr. Vice-President 260 Cherry Hill Rd. Parsippany, NJ 07054
13.	Architect/Engineer	Gilbert Associates, Inc.
14.	Nuclear Steam Supply System	B&W
15.	Constructor	UE&C

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Section Chief	R. R. Keimig
19.	NRC-TMI Support Staff	FTS: 590-3950 D. Haverkamp (/17) 782-3950
20.	Licensing Project Manager	D. C. DiIanni
21.	Docket Number	50-289
22.	License Number/Date of Issue	DPR-50, 4/19/74
23.	Public Document Room	Government Publications Section State Library of Pennsylvania Education Building Commonwealth and Walnut Street Harrisburg, Pennsylvania 17126

INSPECTION STATUS

ENFORCEMENT STATUS

SECTION 2.1 OF THE TMI UNIT 1, ETS, LIMITING CONDITIONS FOR OPERATION REQUIRES, IN PART, THAT DURING THE PERIOD BETWEEN OCTOBER 1 AND MARCH 31, THE PLANT DISCHARGE TEMPERATURES BE NO GREATER THAN 12 DEGREES F ABOVE INLET TEMPERATURE. CONTRARY TO THESE REQUIREMENTS ON DECEMBER 4, 1978 THE PLANT DISCHARGE TEMPERATURE WAS GREATER THAN 12 DEGREES F ABOVE THE PLANT INLET TEMPERATURE.
(7923 DEFICIENCY)

SECTION 4.4.A AND TABLE 3 OF THE UNIT 1 ETS REQUIRE IN PART, THAT MILK SAMPLES BE COLLECTED AND ANALYZED FROM 4 INDICATOR AND 1 BACKGROUND STATION AT MONTHLY INTERVALS DURING THE GRAZING SEASON. CONTRARY TO THESE REQUIREMENTS, MILK SAMPLES WERE NOT COLLECTED AND ANALYZED AT THE REQUIRED FREQUENCIES AND LOCATIONS FOR THE INTERVALS INCLUDING: TM-M-4B1 MARCH, APRIL AND DURING THE WEEK OF MAY 17, 1978; TM-M-5B1 MARCH, APRIL, AND DURING THE WEEK OF MAY 17, 1978; TM-M-14C1 FEBRUARY, AND DURING THE WEEK OF JUNE 7, 1978; TM-M-2G1 DURING THE WEEK OF JUNE 7, 1978; TM-MG-1B1 DURING THE WEEK OF MAY 17, 1978. THIS RESULTED IN FEWER THAN THE REQUIRED NUMBER OF MILK SAMPLES BEING COLLECTED AND ANALYZED DURING THESE TIME INTERVALS.
(7923 DEFICIENCY)

SECTION 4.4.A AND TABLE 3 OF THE UNIT 1 ETS REQUIRE, IN PART, THAT ALL SAMPLES BE ANALYZED WITH SUFFICIENT SENSITIVITIES TO MEET THE MINIMUM SPECIFIED ANALYTICAL SENSITIVITIES. CONTRARY TO THESE REQUIREMENTS, AIR SAMPLES WERE NOT ANALYZED WITH SUFFICIENT ANALYTICAL SENSITIVITY TO MEET THE SPECIFIED MINIMUM SENSITIVITIES FOR I-131 AT THE FOLLOWING LOCATIONS ON SEVERAL OCCASIONS INCLUDING: TM-AI-1S2 FEBRUARY 1-9, MARCH 1-15, JUNE 21-29, AND JULY 6-12, 1978; TM-AI-5A1 FEBRUARY 1-9, 15-22, MARCH 1-5, MAY 25-31, JUNE 21, 29, JULY 6-12, AND NOVEMBER 8-15, 1978; TM-AI-8C1 FEBRUARY 1-9, FEBRUARY 15 - MARCH 15, JUNE 7-14, JUNE 21-JULY 12, AND JULY 26 - AUGUST 2, 1978; TM-AI-15G1 FEBRUARY 1-9, 15-22, MARCH 1-9, MAY 3-11, JUNE 7-14, JUNE 29-JULY 12, AND OCTOBER 11-18, 1978.
(7923 DEFICIENCY)

SECTION 4.4.A AND TABLE 3 OF THE UNIT 1 ENVIRONMENTAL TECHNICAL SPECIFICATIONS REQUIRE, IN PART, THAT SPECIFIED NUMBERS OF SAMPLES OF AIR PARTICULATES AND AIR IODINES BE COLLECTED AND ANALYZED ON A WEEKLY BASIS. (1) CONTRARY TO THESE REQUIREMENTS, THE FOLLOWING ENVIRONMENTAL AIR PARTICULATES SAMPLES WERE NOT COLLECTED AND ANALYZED WEEKLY FOR GROSS BETA ACTIVITY: TM-AP-7F1 JANUARY 11 - FEBRUARY 22, 1978; TM-AP-1S2 APRIL 13 - MAY 31, 1978; TM-AP-7F1 AUGUST 25 - SEPTEMBER 1, 1979; TM-AP-7F1 SEPTEMBER 8 - SEPTEMBER 29, 1979; TM-AP-12B1 SEPTEMBER 29 - OCTOBER 20, 1979. THESE FAILURES RESULTED IN FEWER THAN THE REQUIRED NUMBER OF SAMPLES BEING COLLECTED AND ANALYZED FOR THESE TIME INTERVALS. (2) CONTRARY TO THESE REQUIREMENTS ENVIRONMENTAL AIR IODINE SAMPLES WERE NOT COLLECTED AND ANALYZED WEEKLY AT LOCATION TM-AI-1S2 FROM APRIL 13 TO MAY 31, 1978. THIS RESULTED IN FEWER THAN THE REQUIRED NUMBER OF SAMPLES BEING COLLECTED DURING THIS TIME INTERVAL.
(7923 INFRACTION)

SECTION 4.4.1 AND TABLE 3 OF THE UNIT 1 ETS REQUIRE, IN PART, THAT WATER SAMPLES BE COLLECTED AT THE CITY OF COLUMBIA AT THE SPECIFIED FREQUENCIES AND THE SPECIFIED ANALYSES BE PERFORMED. CONTRARY TO THESE REQUIREMENTS, WATER SAMPLES FROM THE CITY OF COLUMBIA WERE NOT COLLECTED AND ANALYZED AT THE SPECIFIED FREQUENCIES FROM JANUARY 19 THROUGH APRIL 13, 1978. THIS RESULTED IN FEWER THAN THE REQUIRED NUMBER OF SAMPLES BEING COLLECTED AND ANALYZED DURING THIS TIME INTERVAL.
(7923 DEFICIENCY)

SECTION 5.7.1 OF THE UNIT 1 ETS REQUIRES IN PART, THAT RECORDS DEMONSTRATING COMPLIANCE WITH THE LIMITING CONDITIONS FOR OPERATION IN SECTION 2 OF THE ETS, INCLUDING PLANT DELTA T RECORDS, BE RETAINED FOR THE LIFE OF THE PLANT. CONTRARY TO THIS REQUIREMENT, RECORDS OF DELTA T MEASUREMENTS WERE NOT RETAINED FOR FEBRUARY 19, 1979. SECTION 5.8.6 OF THE UNIT 2 ETS REQUIRES, IN PART, THAT RECORDS OF ALL DATA FROM ENVIRONMENTAL MONITORING, SURVEILLANCE AND STUDY ACTIVITIES REQUIRED BY THESE ETS SHALL BE MADE AND RETAINED THROUGHOUT THE TERM OF THE OPERATING LICENSE. CONTRARY TO THIS REQUIREMENT, RECORDS OF PLANT DELTA T, INFLUENT AND EFFLUENT TEMPERATURES, MONITORING REQUIRED BY THE UNIT 2 ETS, WERE NOT RETAINED FOR MARCH 20, 21, 22, 23 AND 24, 1979, DURING WHICH TIME THE UNIT 2 CONTROL ROOM RECORDER WAS OFF-SCALE.
(7923 DEFICIENCY)

SEE CONTINUATION PAGE

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

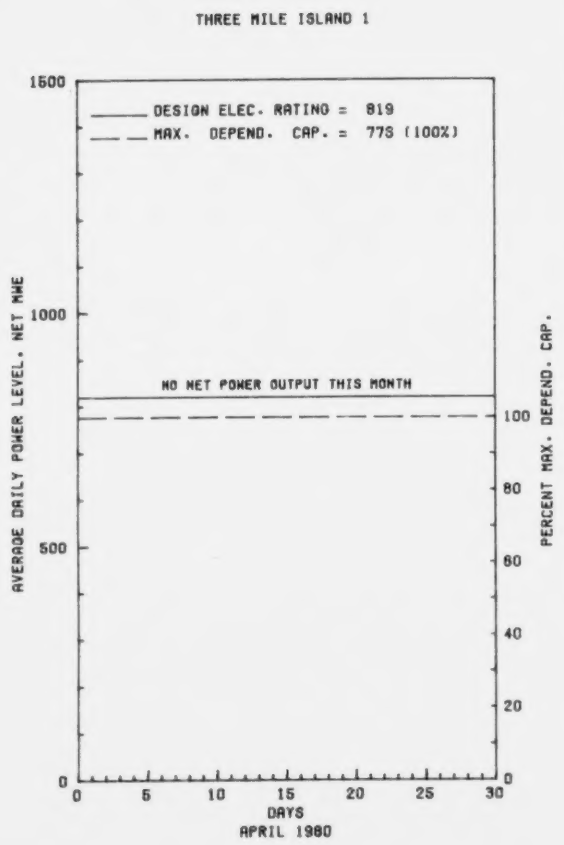
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-001/ DER	02/13/80	03/03/80	Total oil and grease discharged from the IWTS was estimated to be 119.1 pounds, which exceeded the 48.1 pounds/day limit (NPDES permit #9920 noncompliance notification)
80-001/ 01T	01/23/80	02/06/80	Potential overloading conditions that could exist on the 1P 480V bus during certain failures and conditions
80-002/ DER	02/21/80	03/03/80	Total oil and grease discharged from the IWTS was estimated to be 212.1 pounds which exceeded the 48.1 pounds/day limit (NPDES permit #9920 noncompliance notification)
80-002/ 03L	01/31/80	02/29/80	Annual inspection of diesel generator 1A was not performed due to nonavailable parts to support inspection and requirement to operate the "A" decay loop during "B" loop weld repairs
80-003/ 01T	02/06/80	02/29/80	Makeup pump discharge check valves MU-V-73A/C found to have damaged locking devices on the valve seat holddown bolts
80-004/ 01T	03/06/80	03/20/80	During piping reanalysis performed as required by IE Bulletin 79-14, errors were discovered in the seismic analysis of the common decay heat suction piping near valves CH-V1/DH-V2
80-005/ 99X	03/21/80	03/27/80	Anomaly in narrow range pressure instrument-rosemount 1152 pressure transmitters (Special report)
80-006/ 01T	03/25/80	04/08/80	During reactor building local leak rate test of industrial cooler containment isolation valve RB-V7, the updated total leakage exceeded 0.6 LA

REVIEWED BY OMPA LOEB: P E Bobe **DATE:** 6/12/80

REVIEWED BY NRR DOR: **DATE:** 6/16/80

THREE MILE ISLAND 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-289
2. Utility Contact: D. G. Mitchell (215) 921-6579
3. Licensed Thermal Power (MWt): 2535
4. Nameplate Rating (Gross MWe): 968 x 0.9 = 871
5. Design Electrical Rating (Net MWe): 819
6. Maximum Dependable Capacity (Gross MWe): 840
7. Maximum Dependable Capacity (Net MWe): 776
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	719.0	2,903.0	49,632.0
12. Hours Reactor Was Critical	.0	.0	31,731.8
13. Reactor Reserve Shutdown Hours	.0	.0	838.5
14. Hours Generator On-Line	.0	.0	31,180.9
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	0	76,531,071
17. Gross Electrical Energy (MWH)	0	0	25,484,330
18. Net Electrical Energy (MWH)	0	0	23,840,053
19. Unit Service Factor	.0	.0	62.8
20. Unit Availability Factor	.0	.0	62.8
21. Unit Capacity Factor (MDC Net)	.0	.0	61.9
22. Unit Capacity Factor (DER Net)	.0	.0	58.6
23. Unit Forced Outage Rate	100.0	100.0	26.2

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
1	02/17/79	F	719.0	D	4				REGULATORY RESTRAINT ORDER.

SUMMARY

THE UNIT REMAINS SHUTDOWN DUE TO REGULATORY CONSIDERATION OF THE TMI-2 ACCIDENT.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

ENFORCEMENT STATUS

SECTION 5.3 OF THE TMI UNIT 1 ETS REQUIRES, IN PART, THAT INDEPENDENT AUDIT AND REVIEW FUNCTIONS FOR ENVIRONMENTAL MATTERS WILL BE PERFORMED UNDER THE DIRECTION AND CONTROL OF THE MANAGER-GENERATION ENGINEERING. INDEPENDENT REVIEW OF ENVIRONMENTAL MATTERS AND AUDITING OF STATION ACTIVITIES RELATING TO THESE ENVIRONMENTAL TECHNICAL SPECIFICATIONS WILL BE CONDUCTED BY THE RADIATION SAFETY AND ENVIRONMENTAL ENGINEERING SECTION, REPORTING TO THE MANAGER-GENERATION ENGINEERING. THEIR REVIEW WILL BE AUDITED BY OR UNDER THE DIRECTION OF THE MANAGER-GENERATION ENGINEERING. THESE AUDITS AND REVIEWS WILL ENCOMPASS, AMONG OTHER THINGS, THE RESULTS OF THE ENVIRONMENTAL MONITORING PROGRAMS PRIOR TO THEIR SUBMITTAL IN EACH ANNUAL ENVIRONMENTAL MONITORING REPORT. CONTRARY TO THESE REQUIREMENTS, THE RESULTS OF THE ENVIRONMENTAL MONITORING PROGRAMS REPORTED TO THE NRC IN THE 1978 ANNUAL ENVIRONMENTAL MONITORING REPORTS WERE NOT AUDITED.
(7923 INFRACTION)

SECTION 5.5 OF THE ETS FOR TMI UNIT 1 REQUIRES THAT, "DETAILED WRITTEN PROCEDURES, INCLUDING APPLICABLE CHECKLISTS AND INSTRUCTIONS, SHALL BE PREPARED AND FOLLOWED TO IMPLEMENT THE ENVIRONMENTAL TECHNICAL SPECIFICATIONS. PROCEDURES SHALL INCLUDE SAMPLING, DATA RECORDING AND STORAGE, INSTRUMENT CALIBRATION, MEASUREMENTS AND ANALYSES, AND ACTIONS TO BE TAKEN WHEN LIMITS ARE EXCEEDED." (1) THE UNIT 1 RADIOLOGICAL ENVIRONMENTAL MONITORING PROGRAM (REMP) SURVEILLANCE PROCEDURE GP 1402, REV. 0, DATED NOVEMBER 15, 1977, REQUIRES IN PART, THAT REMP SAMPLE COLLECTION SHEETS BE COMPLETED AND DISTRIBUTED AS SPECIFIED IN APPENDIX A OF THAT PROCEDURE. CONTRARY TO THESE REQUIREMENTS, THE REMP SAMPLE COLLECTION, SHEETS DESCRIBED IN THE REMP SURVEILLANCE PROCEDURE GP 1402, REV. 0, WERE NOT COMPLETED AND DISTRIBUTED AS REQUIRED BY APPENDIX A OF THE PROCEDURE FROM APRIL 1979 TO THE DATES OF INSPECTION. (2) UNIT 1 PROCEDURE NO. IC39 REQUIRES ANNUAL CALIBRATION OF THE PLANT INTAKE/DISCHARGE TEMPERATURE MONITORING SYSTEM. CONTRARY TO THESE REQUIREMENTS, THE PLANT INTAKE/DISCHARGE TEMPERATURE MONITORING SYSTEM WAS NOT CALIBRATED DURING 1978 AND 1979 AS REQUIRED.
(7923 INFRACTION)

SECTION 5.5 OF THE UNIT 1 ETS REQUIRES, IN PART, THAT DETAILED WRITTEN PROCEDURES, INCLUDING APPLICABLE CHECKLISTS AND INSTRUCTIONS, SHALL BE PREPARED AND FOLLOWED TO IMPLEMENT THE ENVIRONMENTAL TECHNICAL SPECIFICATIONS. PROCEDURES SHALL INCLUDE SAMPLING, DATA RECORDING AND STORAGE, INSTRUMENT CALIBRATION, MEASUREMENTS AND ANALYSES, AND ACTIONS TO BE TAKEN WHEN LIMITS ARE EXCEEDED. (1) SECTION 2.2 OF THE UNIT 1 ETS REQUIRES, IN PART, THAT A DETERMINATION OF THE PH OF THE CONTENTS OF EACH TANK OF NEUTRALIZED REGENERANT WASTES BE MADE PRIOR TO RELEASE, USING INSTALLED INSTRUMENTATION; THAT ALL NECESSARY ADJUSTMENTS TO MEET THE SPECIFICATION BE MADE PRIOR TO THE INITIATION OF THE RELEASE; AND THAT IF THE INSTALLED INSTRUMENTATION IS OUT OF SERVICE, THE NECESSARY ANALYSES BE PERFORMED USING LABORATORY INSTRUMENTATION PRIOR TO INITIATING THE DISCHARGE. AN ANALYSIS FOR PH IS REQUIRED TO BE PERFORMED ON A SAMPLE TAKEN FROM THE PLANT RIVER WATER DISCHARGE DURING THE RELEASE OF EACH TANK OF REGENERANT WASTES, OR AT WEEKLY INTERVALS AS A MINIMUM FREQUENCY. CONTRARY TO THESE REQUIREMENTS, AS OF THE DATES OF INSPECTION, THE REGENERANT NEUTRALIZING TANK AND THE PLANT RIVER WATER DISCHARGE SAMPLING PROCEDURES WERE NOT PREPARED AS REQUIRED. (2) SECT 2.1.6 OF THE UNIT 1 ETS REQUIRES, IN PART, THAT IF THE UNIT 1 CONTROL ROOM DELTA T CHART RECORDER IS OUT OF SERVICE, THE PLANT INLET AND DISCHARGE TEMPERATURES BE OBTAINED FROM THE RECORDERS LOCATED IN THE MECHANICAL DRAFT COOLING TOWERS (MDCT). SECTIONS 5.5.1 AND 5.5.2 OF THE UNIT 1 ETS REQUIRE IN PART, THAT DETAILED WRITTEN PROCEDURES BE PREPARED AND FOLLOWED TO ENSURE COMPLIANCE WITH THE LIMITING CONDITIONS FOR OPERATION, SECTION 2 OF THE ETS, INCLUDING PROCEDURES TO BE FOLLOWED IN THE EVENT THAT THE UNIT 1 CONTROL ROOM PLANT DELTA T CHART RECORDER IS OUT OF SERVICE. CONTRARY TO THESE REQUIREMENTS, AS OF THE DATES OF THIS INSPECTION, WRITTEN PROCEDURES DETAILING ACTIONS TO BE TAKEN WHEN THE CONTROL ROOM DELTA T RECORDER IS OUT OF SERVICE WERE NOT PREPARED AND FOLLOWED AS REQUIRED.
(7923 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS:

- Shutdown by NRC order pending completion of modifications and other actions related to the TMI-2 accident.

- LAST IE SITE INSPECTION DATE: 4/30/80

- INSPECTION REPORT NO: 50-289/80-09

SUMMARY

INSPECTION SUMMARY

- 50-289/79-23 - Nov 6-9, 15-16, 27-28: Special, unannounced inspection of the environmental monitoring programs for the pre- and post March 1979 TMI-2 accident periods. The inspection included the management controls for the environmental monitoring programs; implementation of the environmental monitoring programs - radiological; implementation of the environmental monitoring programs - biological/ecological; and nonradioactive effluent release rates and limits. The inspection involved 60 hours onsite by two region-based NRC inspectors. Noncompliances: Nine (failure to perform the required audits of the environmental monitoring programs; failure to follow procedures; failure to collect and analyze required drinking water samples; failure to collect and analyze environmental air samples; failure to collect and analyze required milk samples; failure to meet the required detection sensitivities for I-131 in air and water samples; exceeding the plant delta-T limit; failure to maintain plant records; failure to have adequate procedures).

- 50-289/79-24 - Nov 28 - Jan 4: Special inspection by NRC/TMI staff of: plant operations including shift logs and operating records, plant tours and status meetings. The inspection included shift coverage of selected activities in progress including shutdown evolutions involving the movement or processing of radioactive material. Noncompliances: None.

- 50-289/80-01 - Jan 5-31: Special inspection by NRC/TMI staff of licensee action on previous inspection findings and plant operations including shift activities as monitored by NRC shift inspectors. The inspection included day shift coverage with selected backshift coverage by NRC shift radiation specialists. Noncompliances: None.

THREE MILE ISLAND 2

NRC POWER RESTRICTIONS: SEE "LICENSE STATUS"

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Pennsylvania/Dauphin 10 miles SE of Harrisburg
2.	Electric Reliability Council	Mid-Atlantic Area Council
3.	Type of Reactor	PWR
4.	Licenses Power Level (MWT)	2772
5.	Design Electrical Rating (MWE Net)	906
6.	Date of Initial Criticality	3/28/78
7.	Date of Commercial Operation	12/30/78
8.	Condenser Cooling Method	CLOSED
9.	Condenser Cooling Water Source	Susquehanna River

Utility & Contractor Information

10.	Licensee	Metropolitan Edison Company
11.	Corporate Address	P. O. Box 542 Reading, PA 19603
12.	Corporate Contact	R. C. Arnold, Sr. Vice-President 260 Cherry Hill Rd. Parsippany, NJ 07054
13.	Architect/Engineer	Burns & Roe
14.	Nuclear Steam Supply System	B&W
15.	Constructor	UE&C

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Section Chief	A. N. Fasano (717) 782-3950 FTS: 590-3950
19.	IE Resident Inspector	R. Conte (717) 782-3950 FTS: 590-3950
20.	TMI Program Director	B. Snyder
21.	Docket Number	50-320
22.	License Number/Date of Issue	DPR-73/2; 2/8/78
23.	Public Document Room	Government Publications Section State Library of Pennsylvania Education Building Commonwealth & Walnut Streets Harrisburg, Pennsylvania 17126

INSPECTION STATUS

ENFORCEMENT STATUS

SECTION 3.2 AND TABLE 3.2-2 OF THE UNIT 2 ETS REQUIRE, IN PART, THAT MILK SAMPLES BE COLLECTED AND ANALYZED FROM 3 INDICATOR AND 1 BACKGROUND STATION AT SEMI-MONTHLY INTERVALS DURING THE GRAZING SEASON AND MONTHLY INTERVALS AT ALL OTHER TIMES. CONTRARY TO THESE REQUIREMENTS, MILK SAMPLES WERE NOT COLLECTED AND ANALYZED AT THE REQUIRED FREQUENCIES AND LOCATIONS FOR THE INTERVALS INCLUDING: TM-M-4B1 MARCH, APRIL AND DURING THE WEEK OF MAY 17, 1978; TM-M-5B1 MARCH, APRIL, AND DURING THE WEEK OF MAY 17, 1978; TM-M-14C1 FEBRUARY, AND DURING THE WEEK OF JUNE 7, 1978; TM-M-2G1 DURING THE WEEK OF JUNE 7, 1978; TM-MG-1B1 DURING THE WEEK OF MAY 17, 1978. THIS RESULTED IN FEWER THAN THE REQUIRED NUMBER OF MILK SAMPLES BEING COLLECTED AND ANALYZED DURING THESE TIME INTERVALS.
(7928 DEFICIENCY)

SECTION 3.2 AND TABLE 3.2-3 OF THE UNIT 2 ETS REQUIRE, IN PART, THAT ALL SAMPLES BE ANALYZED WITH SUFFICIENT SENSITIVITIES TO MEET THE MINIMUM SPECIFIED ANALYTICAL SENSITIVITIES. (1) CONTRARY TO THESE REQUIREMENTS, AIR SAMPLES WERE NOT ANALYZED WITH SUFFICIENT ANALYTICAL SENSITIVITY TO MEET THE SPECIFIED MINIMUM SENSITIVITIES FOR I-131 AT THE FOLLOWING LOCATIONS ON SEVERAL OCCASIONS INCLUDING: TM-AI-152 FEBRUARY 1-9, MARCH 1-15, JUNE 21-29, AND JULY 6-12, 1978; TM-AI-5A1 FEBRUARY 1-9, 15-22, MARCH 1-5, MAY 25-31, JUNE 21-29, JULY 6-12, AND NOVEMBER 8-15, 1978; TM-AI-8C1 FEBRUARY 1-9, FEBRUARY 15-MARCH 15, JUNE 7-14, JUNE 21-JULY 12, AND JULY 26-AUGUST 2, 1978; TM-AI-15G1 FEBRUARY 1-9, 15-22, MARCH 1-8, MAY 3-11, JUNE 7-14, JUNE 29-JULY 12, AND OCTOBER 11-18, 1978. (2) CONTRARY TO THESE REQUIREMENTS FOR UNIT 2, DRINKING WATER SAMPLES WERE NOT ANALYZED WITH SUFFICIENT ANALYTICAL SENSITIVITY TO MEET THE SPECIFIED MINIMUM SENSITIVITY FOR I-131 DURING MAY AND JUNE 1978.
(7928 DEFICIENCY)

SECTION 3.2 AND TABLE 3.2-2 OF THE UNIT 2 ETS REQUIRE, IN PART, THAT SPECIFIED NUMBERS OF SAMPLES OF AIR PARTICULATES AND AIR IODINES BE COLLECTED AND ANALYZED ON A WEEKLY BASIS. (1) CONTRARY TO THESE REQUIREMENTS, THE FOLLOWING ENVIRONMENTAL AIR PARTICULATES SAMPLES WERE NOT COLLECTED AND ANALYZED WEEKLY FOR GROSS BETA ACTIVITY: TM-AP-7F1 JANUARY 11 - FEBRUARY 22, 1978; TM-AP-152 APRIL 13-MAY 31, 1978; TM-AP-7F1 AUGUST 25-SEPTEMBER 1, 1979;TM-AP-7F1 SEPTEMBER 8-SEPTEMBER 29, 1979; TM-AP-12B1 SEPTEMBER 29 - OCTOBER 20, 1979. THESE FAILURES RESULTED IN FEWER THAN THE REQUIRED NUMBER OF SAMPLES BEING COLLECTED AND ANALYZED FOR THESE TIME INTERVALS. (2) CONTRARY TO THESE REQUIREMENTS ENVIRONMENTAL AIR IODINE SAMPLES WERE NOT COLLECTED AND ANALYZED WEEKLY AT LOCATION TM-AI-152 FROM APRIL 13 TO MAY 31, 1978. THIS RESULTED IN FEWER THAN THE REQUIRED NUMBER OF SAMPLES BEING COLLECTED DURING THIS TIME INTERVAL.
(7928 INFRACTION)

SECTION 3.2 AND TABLE 3.2-2 OF THE UNIT 2 ETS REQUIRE, IN PART, THAT WATER SAMPLES BE COLLECTED AT THE CITY OF COLUMBIA AT THE SPECIFIED FREQUENCIES AND THE SPECIFIED ANALYSES BE PERFORMED. CONTRARY TO THESE REQUIREMENTS, WATER SAMPLES FROM THE CITY OF COLUMBIA WERE NOT COLLECTED AND ANALYZED AT THE SPECIFIED FREQUENCIES FROM JANUARY 19 THROUGH APRIL 13, 1978. THIS RESULTED IN FEWER THAN THE REQUIRED NUMBER OF SAMPLES BEING COLLECTED AND ANALYZED DURING THIS TIME INTERVAL.
(7928 DEFICIENCY)

SECTION 5.8.6 OF THE UNIT 2 ETS REQUIRES, IN PART, THAT RECORDS OF ALL DATA FROM ENVIRONMENTAL MONITORING, SURVEILLANCE AND STUDY ACTIVITIES REQUIRED BY THESE ETS SHALL BE MADE AND RETAINED THROUGHOUT THE TERM OF THE OPERATING LICENSE. CONTRARY TO THIS REQUIREMENT, RECORDS OF PLANT DELTA T, INFLUENT AND EFFLUENT TEMPERATURES, MONITORING REQUIRED BY THE UNIT 2 ETS, WERE NOT RETAINED FOR MARCH 20, 21, 22, 23, AND 24, 1979, DURING WHICH TIME THE UNIT 2 CONTROL ROOM RECORDER WAS OFF-SCALE.
(7928 DEFICIENCY)

LICENSE STATUS

By Order for Modification of License dated July 20, 1979, the licensee's authority to operate the facility was suspended and the licensee's authority was limited to maintenance of the facility in the present shutdown cooling mode (44 F.R. 45271).

SEE CONTINUATION PAGE

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
79-015/ 13L	03/28/79	01/15/80	While investigating leakage of the radwaste system station personnel received radiation doses in excess of quarterly limits for whole body dose
79-016/ 03L	04/28/79	01/15/80	Due to equipment failure the fire protection system was down to one operable fire pump for a period of 3 hours
79-017/ 12L	03/29/79	01/15/80	While manipulating valves and handling containers of coolant samples station personnel received extremity exposures in excess of 10 CFR 20 limits
79-018/ 12L	08/28/79	01/15/80	While attempting to isolate a leak from the makeup system station personnel received whole body and extremity exposures in excess of 10 CFR 20 limits
79-020/ 04L	11/01/79	12/21/79	Environmental studies required by Technical Specifications sections 5.5.4.D and 5.7.1 were terminated without notification to NRC
79-021/ 03L	12/05/79	01/15/80	Diesel generator DF-X-1B tripped 20 seconds after a manual start due to low lube oil pressure, cause was attributed to low lube oil level
79-022/ 03L	12/19/79	01/15/80	During repair of NR-P-10 a damaged coupling was discovered, the cause was attributed to badly worn bearings
80-001/ 99X	01/26/80	02/22/80	Generator auxiliaries supply breaker found tripped during cold shutdown operations (Special report)
80-002/ 99X	01/29/80	02/28/80	Emergency diesel generator failed to start due to mechanical failure of speed switch/tachometer (Special report)
80-003/ 99X	01/31/80	03/01/80	Emergency diesel generator tripped during surveillance due to short timing setpoint error not allowing lube oil pressure to reach operating setpoint (Special report)
80-004/ 03L	02/11/80	03/11/80	Failure of test connection on makeup pump (MU-P-1B) resulting in liquid and gaseous releases
80-005/ 01L	02/18/80	03/19/80	One diesel generator out of service when recovery technical specifications became effective. Both diesels required under new technical specifications.
80-006/ 01L	02/15/80	03/19/80	Boric acid mix tank out of specification when recovery tech specs became effective. Greater minimum inventory required under new tech specs
80-007/ 01L	02/22/80	03/24/80	Unavailability of one aux bldg exhaust due to maintenance when interim recovery TS became effective
80-008/ 01L	02/20/80	03/21/80	Inadvertent removal from service of the nuclear services river water pump providing cooling water for the only operable emergency diesel generator
80-009/ 03L	03/05/80	04/05/80	Emergency diesel generator "B" could not be loaded due to incorrect switch alignment. EDG "A" was down for annual maintenance
80-010/ 01L	03/14/80	03/17/80	Failure of containment outer door seals to reseat
80-011/ 99X	03/21/80	03/27/80	Anomaly in narrow range pressure instrument-rosemount 1152 pressure transmitters (Special report)

REVIEWED BY OMPA LOEB: R E Bobe **DATE:** 6/12/80

REVIEWED BY NRR DOR: D. S. Brinkman **DATE:** 6/13/80

THREE MILE ISLAND 2

AVERAGE DAILY POWER LEVEL (MWe)

OPERATING STATUS

1. Docket Number: 50-320
2. Utility Data Prepared By: R. A. Lengel (215) 921-6581
3. Licensed Thermal Power (MWt): 2772
4. Nameplate Rating (Gross MWe): 1068 X 0.9 = 961
5. Design Electrical Rating (Net MWe): 906
6. Maximum Dependable Capacity (Gross MWe): 0
7. Maximum Dependable Capacity (Net MWe): 0
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons

None

9. Power Level To Which Restricted, If Any (Net MWe): None
10. Reasons For Restrictions, If Any

This Month Yr.

11. Hours In Reporting Period
12. Number Of Hours Reactor Was Critical
13. Reactor Reserve Shutdown Hours
14. Hours Generator On-Line
15. Unit Reserve Sh
16. Gro

(MWH)

19

20. U Availability Factor
21. Unit Capacity Factor (Using MDC Net) 1/
22. Unit Capacity Factor (Using DER Net) 1/
23. Unit Forced Outage Rate
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

None

25. If Shut Down At End Of Report Period, Estimated Date of Startup: N/A

NO DECISION ON FUTURE OPERATION
SHUTDOWN INDEFINITELY

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
SEE SUMMARY BELOW.									

SUMMARY

THE UNIT HAS REMAINED SHUTDOWN SINCE THE 03/28/79 ACCIDENT, WHICH CAUSED EXTENSIVE CORE DAMAGE.

AUTHORITY TO OPERATE FACILITY SUSPENDED BY NRC ORDER DATED JULY 20, 1979, WHICH REQUIRES LICENSEE TO MAINTAIN FACILITY IN A SHUTDOWN CONDITION.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

ENFORCEMENT STATUS

SECTION 5.3 OF THE TMI UNIT 2 ETS REQUIRES, IN PART, THAT INDEPENDENT AUDIT AND REVIEW FUNCTIONS FOR ENVIRONMENTAL MATTERS WILL BE PERFORMED UNDER THE DIRECTION AND CONTROL OF THE MANAGER-GENERATION ENGINEERING. INDEPENDENT REVIEW OF ENVIRONMENTAL MATTERS AND AUDITING OF STATION ACTIVITIES RELATING TO THESE ENVIRONMENTAL TECHNICAL SPECIFICATIONS WILL BE CONDUCTED BY THE RADIATION SAFETY AND ENVIRONMENTAL ENGINEERING SECTION, REPORTING TO THE MANAGER-GENERATION ENGINEERING. THEIR REVIEW WILL BE AUDITED BY OR UNDER THE DIRECTION OF THE MANAGER-GENERATION ENGINEERING. THESE AUDITS AND REVIEWS WILL ENCOMPASS, AMONG OTHER THINGS, THE RESULTS OF THE ENVIRONMENTAL MONITORING PROGRAMS PRIOR TO THEIR SUBMITTAL IN EACH ANNUAL ENVIRONMENTAL MONITORING REPORT. CONTRARY TO THESE REQUIREMENTS, THE RESULTS OF THE ENVIRONMENTAL MONITORING PROGRAMS REPORTED TO THE NRC IN THE 1978 ANNUAL ENVIRONMENTAL MONITORING REPORTS WERE NOT AUDITED.
(7928 INFRACTION)

SECTION 3.1.1 OF THE ETS, UNIT 2, REQUIRES, IN PART, THAT A NONROUTINE REPORT, AS SPECIFIED IN SUBSECTION 5.6.2.B, SHALL BE MADE IF THE THERMAL CHARACTERISTICS OF THE DISCHARGE FROM OUTFALL SERIAL NUMBER 001 FAIL TO COMPLY WITH RELEVANT LIMITATIONS PRESCRIBED BY THE COMMONWEALTH OF PENNSYLVANIA AND THE U.S. ENVIRONMENTAL PROTECTION AGENCY IN THE CERTIFICATES AND PERMITS ISSUED TO THE LICENSEE PURSUANT TO THE PROVISION OF SECTION 401 AND 402 OF PL 92-500, AS CITED IN SECTION 5.4 OF THE ETS. CONTRARY TO THESE REQUIREMENTS, ON MARCH 14, 1979 BETWEEN 11:45 A.M. AND 12:40 P.M., THE STATION DELTA T EXCEEDED 12 DEGREES F, THE SPECIFIED THERMAL DISCHARGE DELTA T LIMIT, AND THIS OCCURRENCE WAS NOT REPORTED TO THE NRC.
(7928 INFRACTION)

SECTION 5.5 OF THE UNIT 2 ETS REQUIRES, IN PART, THAT DETAILED WRITTEN PROCEDURES, INCLUDING APPLICABLE CHECKLISTS AND INSTRUCTIONS, SHALL BE PREPARED AND FOLLOWED TO IMPLEMENT THE ENVIRONMENTAL TECHNICAL SPECIFICATIONS. PROCEDURES SHALL INCLUDE SAMPLING, DATA RECORDING AND STORAGE, INSTRUMENT CALIBRATION, MEASUREMENTS AND ANALYSES, AND ACTIONS TO BE TAKEN WHEN LIMITS ARE EXCEEDED. SECTION 3.1.1.A OF THE UNIT 2 ETS REQUIRES, IN PART, THAT DURING NORMAL OPERATIONS, THE CONTENTS OF THE HOLDING TANKS FOR THE DEMINERALIZER AND CONDENSATE POLISHER REGENERATION SOLUTIONS SHALL BE SAMPLED PRIOR TO BEING DISCHARGED AND THE PH MEASURED IN ACCORDANCE WITH THE PROCEDURES PREPARED BY THE LICENSEE PER SECTION 5.5. CONTRARY TO THESE REQUIREMENTS, AS OF THE DATES OF THIS INSPECTION, PROCEDURES FOR SAMPLING THE HOLDING TANKS FOR DEMINERALIZER AND CONDENSATE POLISHER REGENERATION WERE NOT PREPARED AND FOLLOWED AS REQUIRED.
(7928 INFRACTION)

SECTION 3.1.2.A(1)(C) OF THE UNIT 2 ETS REQUIRES THAT ICHTHYOFAUNA BE SAMPLED TO ESTABLISH POPULATION ESTIMATES IN SPECIFIC AREAS AND TO DETECT AND ASSESS THE SIGNIFICANCE OF CHANGES IN SPECIES COMPOSITION, RELATIVE ABUNDANCE, SEASONAL AND SPATIAL DISTRIBUTION, CONDITION, AND DIVERSITY OF SPECIES AS RELATED TO TMINS OPERATION.... THIS MONITORING PROGRAM SHALL COMMENCE AT INITIAL ATTAINMENT OF NORMAL OPERATION OF UNIT 2 AND CONTINUE FOR A PERIOD OF AT LEAST THREE (3) YEARS. CONTRARY TO THE ABOVE, THE REQUIRED FISH STUDIES WERE NOT PERFORMED DURING THE PERIOD FROM APRIL 1, 1979 THROUGH NOVEMBER 26, 1979.
(7928 INFRACTION)

SECTION 5.5 OF THE ETS FOR TMI UNIT 2 REQUIRES THAT, "DETAILED WRITTEN PROCEDURES, INCLUDING APPLICABLE CHECKLISTS AND INSTRUCTIONS, SHALL BE PREPARED AND FOLLOWED TO IMPLEMENT THE ENVIRONMENTAL TECHNICAL SPECIFICATIONS. PROCEDURES SHALL INCLUDE SAMPLING, DATA RECORDINGS AND STORAGE, INSTRUMENT CALIBRATION, MEASUREMENTS AND ANALYSES, AND ACTIONS TO BE TAKEN WHEN LIMITS ARE EXCEEDED." (1) THE UNIT 2 RADIOLOGICAL ENVIRONMENTAL MONITORING PROGRAM (REMP) SURVEILLANCE PROCEDURE GP 1402, REV. 0, DATED NOVEMBER 15, 1977, REQUIRES IN PART, THAT REMP SAMPLE COLLECTION SHEETS BE COMPLETED AND DISTRIBUTED AS SPECIFIED IN APPENDIX A OF THAT PROCEDURE. CONTRARY TO THESE REQUIREMENTS, THE REMP SAMPLE COLLECTION SHEETS DESCRIBED IN THE REMP SURVEILLANCE PROCEDURE GP 1402, REV. 0, WERE NOT COMPLETED AND DISTRIBUTED AS REQUIRED BY APPENDIX A OF THE PROCEDURE FROM APRIL 1979 TO THE DATES OF INSPECTION. (2) UNIT 2 OPERATING PROCEDURE NO. 2104-2.11 REQUIRES THAT A RELEASE PERMIT BE COMPLETED FOR EACH NEUTRALIZING TANK DISCHARGE, WHICH IS TO INCLUDE SPECIFIED DATA AT THE START AND TERMINATION OF EACH TANK DISCHARGE. CONTRARY TO THESE REQUIREMENTS, THE NEUTRALIZING TANK RELEASE PERMITS WERE NOT COMPLETED, AS REQUIRED, FOR UNIT 2 NEUTRALIZING TANK DISCHARGE ON SEVERAL OCCASIONS, INCLUDING ON OCTOBER 5, 1978, MARCH 1, 2, 3, 4, 5, 6, 7, 8, AND 10, 1979. (3) UNIT 2 PROCEDURE NO. 2104-3.8, REV. 6 ASSIGNS TO UNIT 1 THE RESPONSIBILITY TO ASSURE COMPLIANCE WITH PLANT DELTA T REQUIREMENTS, INCLUDING CALIBRATIONS OF THE PLANT INTAKE/DISCHARGE TEMPERATURE MONITORING SYSTEM. CONTRARY TO THESE REQUIREMENTS, UNIT 2 PROCEDURE 2104-3.8, REV. 6 WAS NOT FULLY IMPLEMENTED IN THAT THE PLANT TEMPERATURE MONITORING SYSTEM WAS NOT CALIBRATED DURING 1978 AND 1979.
(7928 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS:

No significant information to report.

♦ LAST IE SITE INSPECTION DATE: 4/26/80

♦ INSPECTION REPORT NO: 50-320/80-05

SUMMARY

INSPECTION SUMMARY

♦ 50-320/79-28 - Nov 6-9, 15-16, 27-28: Special, unannounced inspection of the environmental monitoring programs for pre- and post-March 1979 TMI-2 periods. The inspection included the management controls for the environmental monitoring programs; implementation of the environmental monitoring programs - radiological; implementation of the environmental monitoring programs - biological/ecological; and nonradioactive effluent release rates and limits. The inspection involved 60 hours onsite by two region-based NRC inspectors. Noncompliances: Ten (failure to perform the required audits of the environmental monitoring program; failure to follow procedures; failure to collect and analyze required drinking water samples; failure to collect and analyze environmental air samples; failure to collect and analyze required milk samples; failure to meet the required detection sensitivities for I-131 in air and water samples; failure to perform the required fish studies; failure to report exceeding the plant delta T limit; failure to maintain plant records; failure to have adequate procedures).

♦ 50-320/80-01 - Jan 5-31: Special inspection by NRC/TMI staff of licensee action on previous inspection findings; new and revised procedures; plant operations including shift activities as monitored by NRC shift inspectors; casualty drills; and contaminated TLDs. The inspection included day shift coverage with selected backshift coverage by NRC shift radiation specialists. Noncompliances: None.

TROJAN

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Oregon/Columbia 42 miles N of Portland
2.	Electric Reliability Council	Western Systems Coordinating Council
3.	Type of Reactor	PWR
4.	Licenses Power Level (MWT)	3411
5.	Design Electrical Rating (MWE Net)	1130
6.	Date of Initial Criticality	December 15, 1975
7.	Date of Commercial Operation	May 20, 1976
8.	Condenser Cooling Method	Natural draft cooling tower
9.	Condenser Cooling Water Source	Columbia River

Utility & Contractor Information

10.	Licensee	Portland General Electric Co.
11.	Corporate Address	121 S. W. Salmon St. Portland, Oregon 97204
12.	Corporate Contact	C. Goodwin, Jr., Asst. V.P.
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Multiple

Regulatory Information

16.	IE Region responsible	V
17.	IE Branch Chief	J. L. Crews
18.	IE Senior Inspector	D. M. Sternberg
19.	IE Resident Inspector	M. H. Malmros (503) 556-3751
20.	Licensing Project Manager	C. M. Trammell
21.	Docket Number	50-344
22.	License Number/Date of Issue	NPF-1, 11/21/75
23.	Public Document Room	Columbia County Courthouse Law Library Circuit Court Room St. Helens, Oregon 97501

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

SYSTEMS AND COMPONENT PROBLEMS:

None.

FACILITY ITEMS (PLANS AND PROCEDURES):

+ None.

MANAGERIAL ITEMS:

None.

PLANT STATUS

+ The plant commenced a refueling and maintenance outage on April 11, 1980. Startup is scheduled for June 1980.

LAST IE SITE INSPECTION DATE: 04/01-31/80+

INSPECTION REPORT NO: 50-344/80-10+

SUMMARY

INSPECTION SUMMARY:

+ Inspection on March 17-21 & April 21, 1980 (Report No. 50-344/80-05)
Areas Inspected: Included a review of security plan; security program audit; security organization; protected area physical barriers; alarm stations; detection aids; communications; testing and maintenance; power supply; and locks, keys and combinations. The inspection involved 136 inspector hours onsite by four NRC inspectors.

Results: The licensee was found to be in compliance with NRC requirements within the ten areas examined except for the following item (Infraction - Failure to close and lock communications room door).

+ Inspection on March 3-31, 1980 (Report No. 50-344/80-06)
Areas Inspected: Routine inspections of plant operations, maintenance, radiological protection, physical security, review and audit, and followup on Licensee Event Reports. The inspection involved 144 inspector hours by the NRC Resident Inspectors.

Results: No items of noncompliance or deviations were identified.

+ Inspection on April 14-18, 1980 (Report No. 50-344/80-07)
Report being prepared; to be reported next month.

+ Inspection on April 8, 1980 (Report No. 50-344/80-08)
Report being prepared; to be reported next month.

+ Inspection on April 21-24, 1980 (Report No. 50-344/80-09)

Report being prepared; to be reported next month.

+ Inspection on April 1-30, 1980 (Report No. 50-344/80-10)
Areas Inspected: Routine inspections of plant operations, maintenance, surveillance testing, physical security, review and audit, refueling activities, and followup on Licensee Event Reports. The inspection involved 206 inspector hours by the NRC Resident Inspectors.

Results: No items of noncompliance or deviations were identified.

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-02/ 03L-0	02-07-80	03-07-80	Broken cooling water hose on diesel driven auxiliary feedwater pump.
80-04/ 03L-0	03-09-80	04-07-80	Inadequate air flow in control room emergency ventilation system.

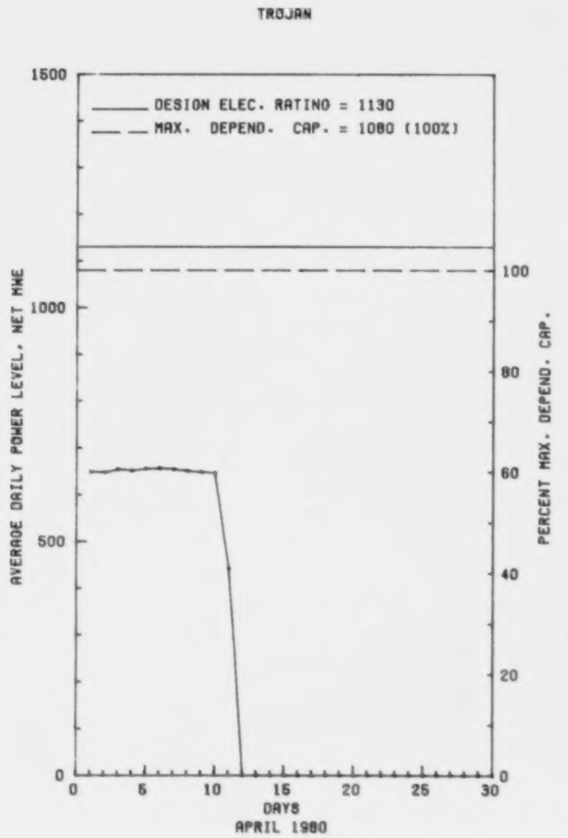
REVIEWED BY OMPA LOEB: PE Sole **DATE:** 6/10/80

REVIEWED BY NRR DOR: CM Tramell **DATE:** 6/13/80

GPO 871-667

TROJAN

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-344
2. Utility Contact: G. G. Bair (503) 556-3713 x234
3. Licensed Thermal Power (MWt): 3411
4. Nameplate Rating (Gross MWe): 1280 x 0.95 = 1216
5. Design Electrical Rating (Net MWe): 1130
6. Maximum Dependable Capacity (Gross MWe): 1122
7. Maximum Dependable Capacity (Net MWe): 1080
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	719.0	2,903.0	32,111.0
12. Hours Reactor Was Critical	257.7	2,422.0	18,720.0
13. Reactor Reserve Shutdown Hours	.0	.0	2,171.8
14. Hours Generator On-Line	257.4	2,416.8	18,044.6
15. Unit Reserve Shutdown Hours	.0	.0	1,508.7
16. Gross Thermal Energy (MWH)	569,119	6,839,434	55,735,044
17. Gross Electrical Energy (MWH)	178,165	2,203,790	18,189,205
18. Net Electrical Energy (MWH)	164,714	2,094,975	17,128,294
19. Unit Service Factor	35.8	83.3	56.2
20. Unit Availability Factor	35.8	83.3	60.9
21. Unit Capacity Factor (MDC Net)	21.2	66.8	49.4
22. Unit Capacity Factor (DER Net)	20.3	63.9	47.2
23. Unit Forced Outage Rate	.0	1.0	27.7

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 06/01/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-04	04/11/80	S	441.6	C	1		RC	FUELXX	REFUELING OUTAGE.

SUMMARY

THE UNIT WAS SHUTDOWN ON 04/11/80 FOR REFUELING.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

TURKEY POINT 3

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Florida/Dade 25 miles S of Miami
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2200
5.	Design Electrical Rating (MWE Net)	693
6.	Date of Initial Criticality	October 20, 1972
7.	Date of Commercial Operation	December 14, 1972
8.	Condenser Cooling Method	Recirculating Canal System
9.	Condenser Cooling Water Source	Atlantic Ocean

Utility & Contractor Information

10.	Licensee	Florida Power and Light Company
11.	Corporate Address	9250 West Flagler Street P. O. Box 013100 Miami, Florida 33101
12.	Corporate Contact	R. E. Uhrig, V.P. Nuclear Affairs
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. Lewis
18.	IE Section Chief	D. R. QUICK
19.	IE Resident Inspector	R. J. Vogt-Lowell (305) 245-7669
20.	Licensing Project Manager	M. Grotenhuis (301) 492-7128
21.	Docket Number	50-250
22.	License Number/Date of Issue	DPR-31, 7/19/72
23.	Public Document Room	Environmental and Urban Affairs Library Florida International University Miami, Florida 33199

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

NORMAL OPERATION AT 100% POWER.

- LAST IE SITE INSPECTION DATE: APRIL 1-30, 1980
- INSPECTION REPORT NO: 50-250/80-16

SUMMARY

INSPECTION SUMMARY

- INSPECTION MARCH 25-28 (80-08): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 23 INSPECTOR-HOURS ON SITE IN THE AREAS OF QUALITY CONTROL AND CONFIRMATORY MEASUREMENTS, INCLUDING REVIEW OF THE LABORATORY QUALITY CONTROL PROGRAM; REVIEW OF CHEMICAL AND RADIOCHEMICAL PROCEDURES; AND COMPARISON OF THE RESULTS OF SPLIT SAMPLES ANALYZED BY THE LICENSEE AND THE NRC REGION II MOBILE LABORATORY. OF THE THREE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN TWO AREAS; ONE ITEM OF NONCOMPLIANCE WAS FOUND IN ONE AREA (DEFICIENCY - FAILURE TO HAVE AND FOLLOW WRITTEN PROCEDURES FOR RADIOCHEMISTRY COUNTING EQUIPMENT).

INSPECTION MARCY 26-27 (80-09): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 8 INSPECTOR-HOURS ON SITE IN THE AREAS OF PIPE SUPPORT BASEPLATE DESIGNS USING CONCRETE EXPANSION ANCHOR BOLTS (IEB 79-02); AND SEISMIC ANALYSIS FOR AS-BUILT SAFETY-RELATED PIPING SYSTEMS (IEB 79-14). OF THE TWO AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION FEBRUARY 1-29 (80-10): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 67 INSPECTOR-HOURS ON SITE IN THE AREAS OF PLANT NUCLEAR SAFETY COMMITTTEE MEETINGS; PLANT OPERATIONS; FOLLOWUP ON REPORTABLE EVENTS; ADHERENCE TO PLANT PROCEDURES; AND AUXILIARY FEEDWATER PUMP MAINTENANCE PROCEDURES. OF THE FIVE AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE WAS FOUND IN ONE AREA (INFRACTION - FAILURE TO FOLLOW PROCEDURE); ONE APPARENT DEVIATION WAS FOUND IN ONE AREA (DEVIATION - FAILURE TO GENERATE FINAL APPROVED MAINTENANCE PROCEDURES FOR AUXILIARY FEEDWATER).

INSPECTION FEBRUARY 28-29 (80-11): THIS SPECIAL, ANNOUNCED INSPECTION INVOLVED 3 INSPECTOR-HOURS ON SITE IN THE AREA OF TRANSPORTATION OF RADIOACTIVE MATERIALS AND CONSISTED OF A REVIEW OF THE CIRCUMSTANCES SURROUNDING A RECENT RADIOACTIVE WASTE SHIPMENT WHICH REPORTEDLY DID NOT CONFORM TO DEPARTMENT OF TRANSPORTATION REGULATIONS. OF THE ONE AREA INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION APRIL 10-11 (80-14): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 12 INSPECTOR-HOURS ON SITE IN THE AREAS OF CONSTRUCTION STATUS, LICENSEE ACTION ON PREVIOUS INSPECTION FINDINGS, AND REVIEW OF STORAGE RECORDS. OF THE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION MARCH 1-31 (80-15): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 37 INSPECTOR-HOURS ON SITE IN THE AREAS OF SURVEILLANCE TEST OBSERVATION; LOW POWER PHYSICS TESTING ACTIVITIES; PLANT OPERATIONS; AND FOLLOWUP OF REPORTABLE EVENTS. OF THE 4 AREAS INSPECTED, NO APPARENT ITEM OF NONCOMPLIANCE WERE IDENTIFIED, BUT ONE APPARENT DEVIATION WAS FOUND IN ONE AREA (DEVIATION - FAILURE TO HAVE INSTALLATION OF FIRE PUMPS IN ACCORDANCE WITH NFPA-20).

REPORTS RECEIVED FROM LICENSEE

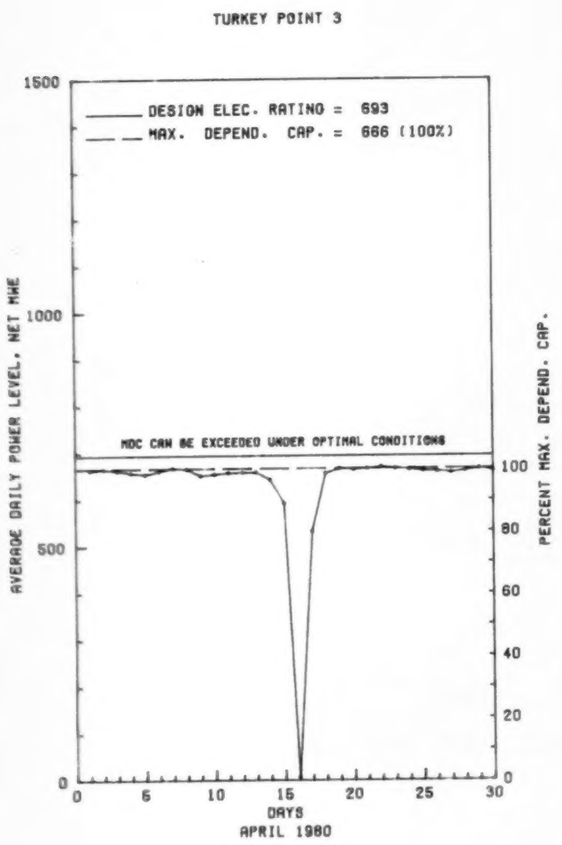
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
80-03/ 03L-0	02/12/80	03/13/80	30 DAY LER	ACTUAL REACTOR POWER GREATER THAN INDICATED POWER.
80-04/ 03L-0	02/25/80	03/26/80	30 DAY LER	EMERGENCY DIESEL GENERATOR RADIATOR LEAK.
80-05/ 03L-0	02/29/80	03/31/80	30 DAY LER	FIRE HOSE STATION HS-AB-02 INOPERABLE DUE TO HOSE TOO SHORT.

REVIEWED BY OMPA LOEB: P E Bole **DATE:** 6/12/80

REVIEWED BY NRR DOR: Don [illegible] **DATE:** 6/13/80

TURKEY POINT 3

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-250
2. Utility Contact: V. T. Chilson (305) 552-3824
3. Licensed Thermal Power (MWt): 2200
4. Nameplate Rating (Gross MWe): 894 x 0.85 = 760
5. Design Electrical Rating (Net MWe): 693
6. Maximum Dependable Capacity (Gross MWe): 700
7. Maximum Dependable Capacity (Net MWe): 666

ITEM 21 CALCULATED BY USING A WEIGHTED AVERAGE

8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	719.0	2,903.0	64,904.6
12. Hours Reactor Was Critical	715.6	2,095.2	49,364.6
13. Reactor Reserve Shutdown Hours	.0	.0	213.3
14. Hours Generator On-Line	696.3	1,946.7	47,636.9
15. Unit Reserve Shutdown Hours	.0	.0	121.8
16. Gross Thermal Energy (MWH)	1,513,229	4,159,227	95,627,861
17. Gross Electrical Energy (MWH)	478,500	1,327,045	30,408,045
18. Net Electrical Energy (MWH)	454,486	1,254,849	28,775,943
19. Unit Service Factor	96.8	67.1	73.4
20. Unit Availability Factor	96.8	67.1	73.6
21. Unit Capacity Factor (MDC Net)	94.9	64.9	67.1
22. Unit Capacity Factor (DER Net)	91.2	62.4	64.0
23. Unit Forced Outage Rate	1.7	2.2	2.7

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
STEAM GEN TUBE INSPEC PROGRAM - OCT. 4, - OCT. 31, 1980
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
9	04/15/80	S	10.8	B	3		HA	VALVEX	UNIT WAS REMOVED FROM SERVICE (DELIBERATE MANUAL TURBINE SHUTDOWN) TO REPAIR INOPERABLE TURBINE CONTROL VALVE SERVOMOTOR TEST VALVE ASSEMBLY.
10	04/16/80	F	11.9	A	9		HC	HTEXCH	UNIT WAS REMOVED FROM SERVICE DUE TO SECONDARY WATER CHEMISTRY LIMITATION CORRECTIVE ACTIONS INCLUDED REPAIRING CONDENSER TUBE LEAKS.

SUMMARY

OPERATION WAS AT NEAR FULL POWER DURING THE REPORT PERIOD. THE UNIT WAS TAKEN OFF LINE ON 04/15/80 TO REPAIR THE TURBINE CONTROL VALVE SURVOMOTOR TEST VALVE ASSEMBLY, AND ON 04/16/80 TO REPAIR CONDENSER TUBE LEAKS.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

TURKEY POINT 4

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Florida/Dade 25 miles S of Miami
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2200
5.	Design Electrical Rating (MWE Net)	693
6.	Date of Initial Criticality	June 11, 1973
7.	Date of Commercial Operation	September 7, 1973
8.	Condenser Cooling Method	Recirculating Canal System
9.	Condenser Cooling Water Source	Atlantic Ocean

Utility & Contractor Information

10.	Licensee	Florida Power and Light Company
11.	Corporate Address	9250 West Flagler Street P. O. Box 013100 Miami, Florida 33101
12.	Corporate Contact	R. E. Uhrig, V. P. Nuclear Affairs
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. Lewis
18.	IE Section Chief	D. R. QUICK
19.	IE Resident Inspector	R. J. Vogt-Lowell (305) 245-7669
20.	Licensing Project Manager	M. Grotenhuis (301) 492-7128
21.	Docket Number	50-251
22.	License Number/Date of Issue	DPR-41, 4/10/73
23.	Public Document Room	Environmental and Urban Affairs Library Florida International University Miami, Florida 33199

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

+ NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

+ UNIT SHUT DOWN FOR STEAM GENERATOR TUBE INSPECTION AND LOW PRESSURE TURBINE ROTOR REPLACEMENT.

+ LAST IE SITE INSPECTION DATE: APRIL 1-30, 1980

+ INSPECTION REPORT NO: 50-251/80-14

SUMMARY

INSPECTION SUMMARY

+ INSPECTION MARCH 25-28 (80-08): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 23 INSPECTOR-HOURS ON SITE IN THE AREAS OF QUALITY CONTROL AND CONFIRMATORY MEASUREMENTS, INCLUDING REVIEW OF THE LABORATORY QUALITY CONTROL PROGRAM; REVIEW OF CHEMICAL AND RADIOCHEMICAL PROCEDURES; AND COMPARISON OF THE RESULTS OF SPLIT SAMPLES ANALYZED BY THE LICENSEE AND THE NRC REGION II MOBILE LABORATORY. OF THE THREE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN TWO AREAS; ONE ITEM OF NONCOMPLIANCE WAS FOUND IN ONE AREA (DEFICIENCY - FAILURE TO HAVE AND FOLLOW WRITTEN PROCEDURES FOR RADIOCHEMISTRY COUNTING EQUIPMENT).

INSPECTION MARCY 26-27 (80-09): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 8 INSPECTOR-HOURS ON SITE IN THE AREAS OF PIPE SUPPORT BASEPLATE DESIGNS USING CONCRETE EXPANSION ANCHOR BOLTS (IEB 79-02); AND SEISMIC ANALYSIS FOR AS-BUILT SAFETY-RELATED PIPING SYSTEMS (IEB 79-14). OF THE TWO AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION FEBRUARY 1-29 (80-10): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 68 INSPECTOR-HOURS ON SITE IN THE AREAS OF PLANT NUCLEAR SAFETY COMMITTEE MEETINGS; PLANT OPERATIONS; FOLLOWUP ON REPORTABLE EVENTS; ADHERENCE TO PLANT PROCEDURES; AND AUXILIARY FEEDWATER PUMP MAINTENANCE PROCEDURES. OF THE FIVE AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE WAS FOUND IN ONE AREA (INFRACTION - FAILURE TO FOLLOW PROCEDURE); ONE APPARENT DEVIATION WAS FOUND IN ONE AREA (DEVIATION - FAILURE TO GENERATE FINAL APPROVED MAINTENANCE PROCEDURES FOR AUXILIARY FEEDWATER).

INSPECTION FEBRUARY 28-29 (80-11): THIS SPECIAL, ANNOUNCED INSPECTION INVOLVED 3 INSPECTOR-HOURS ON SITE IN THE AREA OF TRANSPORTATION OF RADIOACTIVE MATERIALS AND CONSISTED OF A REVIEW OF THE CIRCUMSTANCES SURROUNDING A RECENT RADIOACTIVE WASTE SHIPMENT WHICH REPORTEDLY DID NOT CONFORM TO DEPARTMENT OF TRANSPORTATION REGULATIONS. OF THE ONE AREA INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION APRIL 10-11 (80-12): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 12 INSPECTOR-HOURS ON SITE IN THE AREAS OF CONSTRUCTION STATUS, LICENSEE ACTION ON PREVIOUS INSPECTION FINDINGS, AND REVIEW OF STORAGE RECORDS. OF THE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION MARCH 1-31 (80-13): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 27 INSPECTOR-HOURS ON SITE IN THE AREAS OF SURVEILLANCE TEST OBSERVATION; PLANT OPERATIONS; AND FOLLOWUP OF REPORTABLE EVENTS. OF THE 3 AREAS INSPECTED, NO APPARENT ITEM OF NONCOMPLIANCE WERE IDENTIFIED, BUT ONE APPARENT DEVIATION WAS FOUND IN ONE AREA (DEVIATION - FAILURE TO HAVE INSTALLATION OF FIRE PUMPS IN ACCORDANCE WITH NFPA-20).

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

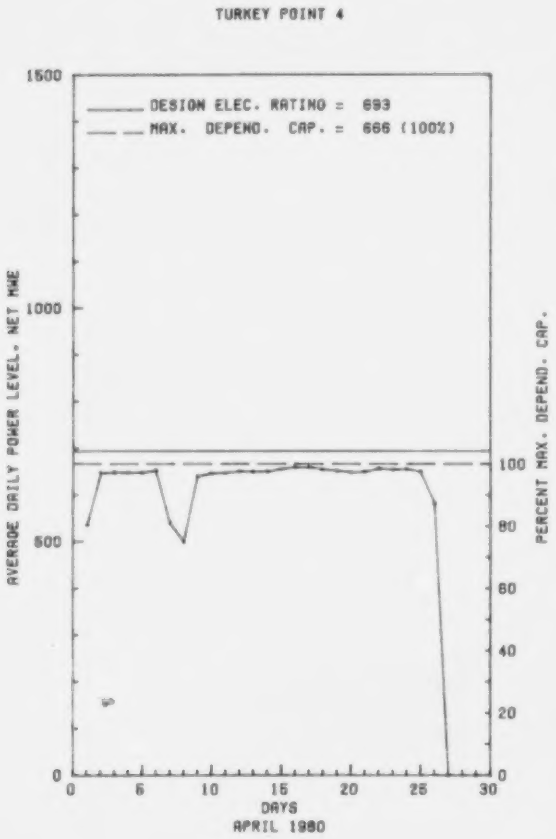
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
80-02/ 03L-0	02/14/80	03/17/80	30 DAY LER	LOW CONDENSATE STORAGE TANK LEVEL DUE TO WATER PLANT OUT OF SERVICE.
80-03/ 03L-0	02/15/80	03/17/80	30 DAY LER	STEAM GENERATOR SAFETY VALVE FAILS TO FULLY RESEAT.
80-04/ 03L-0	02/16/80	03/17/80	30 DAY LER	LOW CONDENSATE STORAGE TANK LEVEL.

REVIEWED BY OMPA LOEB: P E Bobe **DATE:** 6/12/80

REVIEWED BY NRR DOR: M Grotenhuis **DATE:** 6/16/80

TURKEY POINT 4

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-251
2. Utility Contact: V. T. Chilson (305) 552-3824
3. Licensed Thermal Power (MWt): 2200
4. Nameplate Rating (Gross MWe): 894 x 0.85 = 760
5. Design Electrical Rating (Net MWe): 693
6. Maximum Dependable Capacity (Gross MWe): 700
7. Maximum Dependable Capacity (Net MWe): 666
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

ITEM 21 CALCULATED BY USING A WEIGHTED AVERAGE.

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	719.0	2,903.0	58,632.0
12. Hours Reactor Was Critical	623.0	2,666.0	43,493.8
13. Reactor Reserve Shutdown Hours	.0	12.3	166.6
14. Hours Generator On-Line	620.2	2,616.1	41,798.6
15. Unit Reserve Shutdown Hours	.0	12.3	31.2
16. Gross Thermal Energy (MWH)	1,333,984	5,644,654	86,863,634
17. Gross Electrical Energy (MWH)	416,085	1,780,840	27,684,318
18. Net Electrical Energy (MWH)	394,581	1,689,369	26,219,365
19. Unit Service Factor	86.3	90.1	71.3
20. Unit Availability Factor	86.3	90.5	71.3
*21. Unit Capacity Factor (MDC Net)	82.4	87.4	67.6
22. Unit Capacity Factor (DER Net)	79.2	84.0	64.5
23. Unit Forced Outage Rate	.5	.4	3.1

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 06/17/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
11	04/01/80	F	.0	A	5		HB	HTEXCH	UNIT WAS REMOVED FROM SERVICE TO PERFORM SCHEDULED MAINTENANCE AND INSPECTIONS, INCLUDING STEAM GENERATOR TUBE INSPECTION PROGRAM, AND TURBINE L.P. ROTOR CHANGEOUT.
12	04/07/80	F	3.4	A	3		HH	MOTORX	UNIT WAS TRIPPED BY STEAM GENERATOR LEVEL PROTECTION SYSTEM DURING A TRANSIENT CONDITION CAUSED BY THE FAILURE OF CONDENSATE PUMP MOTOR NO. 4A. CORRECTIVE ACTIONS INCLUDED REPAIRING FAILED ELECTRICAL INSULATION ON THE MOTOR.
13	04/26/80	S	95.4	B	1		HC	HTEXCH	LOAD REDUCTION TO REPAIR CONDENSER TUBE LEAK.

SUMMARY

OPERATION WAS AT NEAR FULL POWER UNTIL 04/26/80, WHEN THE UNIT WAS TAKEN OFF LINE FOR SCHEDULED MAINTENANCE AND INSPECTIONS. LOAD WAS REDUCED ON 04/01/80 TO REPAIR A CONDENSER TUBE LEAK, AND A UNIT TRIP OCCURRED ON 04/07/80 BY THE STEAM GENERATOR LEVEL PROTECTION SYSTEM DURING A TRANSIENT CAUSED BY FAILURE OF A CONDENSATE PUMP MOTOR.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

VERMONT YANKEE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Vermont/Windham 5 miles S of Brattleboro
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	1593
5.	Design Electrical Rating (MWE Net)	514
6.	Date of Initial Criticality	March 24, 1972
7.	Date of Commercial Operation	November 30, 1972
8.	Condenser Cooling Method	Closed Cycle Cooling Towers
9.	Condenser Cooling Water Source	Connecticut River

Utility & Contractor Information

10.	Licensee	Vermont Yankee Nuclear Power Corp.
11.	Corporate Address	20 Turnpike Road Westboro, Massachusetts 01581
12.	Corporate Contact	D. W. Edwards Assistant to the Vice President
13.	Architect/Engineer	Ebasco
14.	Nuclear Steam Supply System	GE
15.	Constructor	Ebasco

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	IE Section Chief	T. T. MARTIN
19.	IE Resident Inspector	W. J. RAYMOND
20.	Licensing Project Manager	V. L. Rooney
21.	Docket Number	50-271
22.	License Number/Date of Issue	DPR-28, 3/21/72
23.	Public Document Room	Brooks Memorial Library 224 Main Street Brattleboro, Vermont 05301

INSPECTION STATUS

ENFORCEMENT STATUS

10 CFR 20.201(B) REQUIRES EACH LICENSEE TO MAKE OR CAUSE TO BE MADE SUCH SURVEYS AS MAY BE NECESSARY TO COMPLY WITH THE REGULATORY REQUIREMENTS SPECIFIED IN 10 CFR PART 20. CONTRARY TO THE ABOVE REQUIREMENT SURVEYS ADEQUATE TO ASSURE COMPLIANCE WITH 10 CFR 20.101(A) WERE NOT PERFORMED IN THE REACTOR VESSEL ON 10/2/79. NO MEASUREMENTS OF BETA RADIATION WERE MADE PRIOR TO HYDROLASTING ALTHOUGH SUBSEQUENT MEASUREMENTS INDICATED BETA EXPOSURES OF 1.75 R/HR ON UNHYDROLASED AREAS.
(7914 INFRACTION)

TS 6.5.B REQUIRES IN PART THAT "RADIATION CONTROL STANDARDS & PROCEDURES SHALL BE PREPARED, APPROVED & MAINTAINED...". PROCEDURE AP 0502 REQUIRES THAT STANDARD RADIATION WORK PERMITS (RWP) BE USED FOR NONROUTINE WORK IN THE RADIATION CONTROL AREA (RCA) & THAT PERTINENT RESULTS OF RADIOLOGICAL SURVEYS BE ENTERED ON THE RWP. CONTRARY TO THE ABOVE REQUIREMENT, A NONROUTINE TASK, HYDROLASING THE REACTOR VESSEL, WAS PERFORMED UNDER A ROUTINE RWP WHICH DID NOT INDICATE RADIOLOGICAL DATA PERTINENT TO THE TASK. THE RWP INDICATED CONTAMINATION LEVELS OF 5000-3000 DPM/100 CM TO THE SECOND & RADIATION LEVELS OF 10-35 MR/HR WHEN LICENSEE SURVEYS INDICATED CONTAMINATION LEVELS IN EXCESS OF 1,000,000 DPM/100 CM TO THE SECOND & RADIATION LEVELS IN EXCESS OF 3,000 MR/HR.
(7914 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

- SYSTEMS AND COMPONENT PROBLEMS:
- On May 3, 1980 with the facility operating at full rated power, a lightning strike in the plant yard caused the loss of several annunciator panels on the electrical/turbine sections of the main control board. No other control board instruments were affected. The annunciator panels were returned to a fully operable status on May 5, 1980.

FACILITY ITEMS (PLANS AND PROCEDURES):

- Plant shutdown for refueling operations is scheduled for September 1980.

PLANT STATUS:

- Routine operations at power continue. SJAE offgas release rate is 4470 microcuries per second. Stack release rate is below 100 microcuries per second.
- LAST IE SITE INSPECTION DATE: 5/1/80
- INSPECTION REPORT NO: 50-271/80-06

SUMMARY

INSPECTION SUMMARY

- 50-271/80-04 - Mar 13-14 & 17-19: Routine, unannounced inspection by two region-based inspectors (41 hours onsite) of Small Break LOCA Operator Guidelines, including emergency procedure preparation and review for technical adequacy, operator training and interviews with operators on emergency procedure response; and followup on reportable occurrence LER 80-11/1P, potential for safeguard electrical bus degradation. Noncompliances: None.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

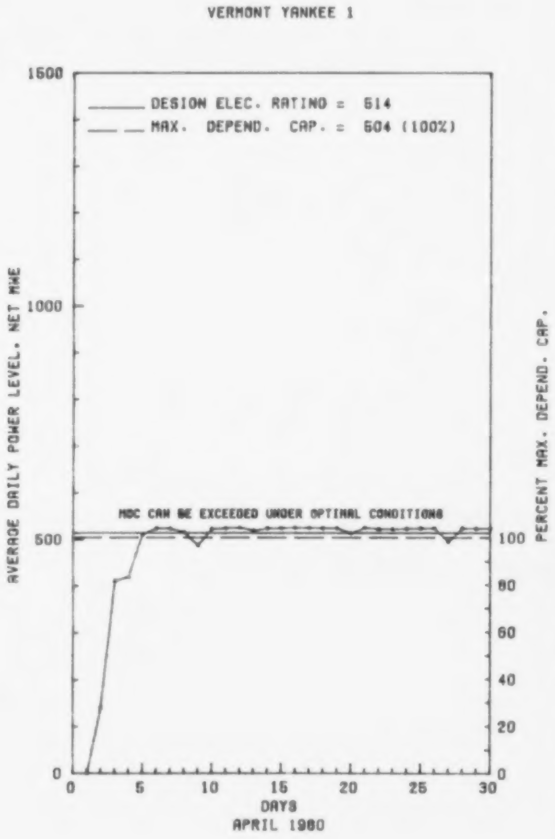
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-010/03L	03/07/80	03/31/80	RPS system response time design value vs TS requirements
80-012/01T	03/28/80	04/11/80	Inadequate pipe supports as a result of IEB-79-14 inspection
80-013/03L	03/26/80	04/14/80	Monthly surveillance identified HPCI turbine steam flow controller failed in auto mode

REVIEWED BY OMPA LOEB: P E Bobe **DATE:** 6/12/80

REVIEWED BY NRR DOR: Dick Clark **DATE:** 06-13-80

VERMONT YANKEE

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-271
2. Utility Contact: F.C. BEERS (617) 366-9011
3. Licensed Thermal Power (MWt): 1593
4. Nameplate Rating (Gross MWe): 626 x 0.9 = 563
5. Design Electrical Rating (Net MWe): 514
6. Maximum Dependable Capacity (Gross MWe): 535
7. Maximum Dependable Capacity (Net MWe): 504
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	719.0	2,903.0	66,697.8
12. Hours Reactor Was Critical	701.1	2,757.3	53,915.9
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	682.5	2,714.6	52,042.4
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,058,111	4,166,247	73,398,858
17. Gross Electrical Energy (MWH)	362,493	1,427,733	24,397,403
18. Net Electrical Energy (MWH)	347,124	1,370,163	23,138,528
19. Unit Service Factor	94.9	93.5	78.0
20. Unit Availability Factor	94.9	93.5	78.0
21. Unit Capacity Factor (MDC Net)	96.0	93.6	68.8
22. Unit Capacity Factor (DER Net)	94.1	91.8	67.5
23. Unit Forced Outage Rate	.0	1.0	6.2

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-9	03/30/80	S	36.5	B	4		RA	CONROD	TURBINE SURVEILLANCE AND CONTROL ROD ADJUSTMENT.
80-10	04/02/80	S	.0	H	5		RA	CONROD	TURBINE SURVEILLANCE & CONTROL ROD ADJUSTMENT.
80-11	04/27/80	S	.0	H	5		HP	VALVEX	MAIN STEAM VALVE WHICH ISOLATES THE STEAM SUPPLY TO THE MAIN TURBINE STEAM SEAL REGULATOR WAS LEAKING STEAM TO THE ATMOSPHERE. VALVE WAS REPLACED WITH PIPING.

SUMMARY

OPERATION WAS AT NEAR FULL POWER FOLLOWING RETURN TO SERVICE ON 04/02/80 FROM AN OUTAGE TO REPLACE THE MAIN STEAM VALVE WITH PIPING. BRIEF LOAD REDUCTIONS WERE NECESSARY ON 04/20/80 AND 04/27/80 FOR TURBINE SURVEILLANCE AND CONTROL ROD ADJUSTMENT.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for License Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

YANKEE ROWE

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Massachusetts/Franklin 25 miles NE of Pittsfield
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	600
5.	Design Electrical Rating (MWE Net)	175
6.	Date of Initial Criticality	August 19, 1960
7.	Date of Commercial Operation	July 1961
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Deerfield River

Utility & Contractor Information

10.	Licensee	Yankee Atomic Electric Company
11.	Corporate Address	20 Turnpike Road Westboro, Massachusetts 01581
12.	Corporate Contact	D. W. Edwards, Asst. to V.P.
13.	Architect/Engineer	Stone & Webster
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Section Chief	T. T. MARTIN
19.	IE Resident Inspector	T. FOLEY
20.	Licensing Project Manager	A. Burger
21.	Docket Number	50-29
22.	License Number/Date of Issue	DPR-3, 12/24/63
23.	Public Document Room	Greenfield Community College 1 College Drive Greenfield, Massachusetts 01301

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

- The LP Turbine is being repaired, with remanufacturing of the turbine blades the limiting item.

FACILITY ITEMS (PLANS AND PROCEDURES):

- Plant startup is tentatively scheduled for mid-September 1980.

MANAGERIAL ITEMS:

- NRR met with representatives of Yankee Atomic Electric Co. on April 29, 1980 to discuss seismic reanalysis, flooding and risk assessment.

PLANT STATUS:

- Shutdown for LP Turbine repair.

LAST IE SITE INSPECTION DATE: 3/17/80

INSPECTION REPORT NO: 50-29/80-07

SUMMARY

INSPECTION SUMMARY

- None

REPORTS RECEIVED FROM LICENSEE

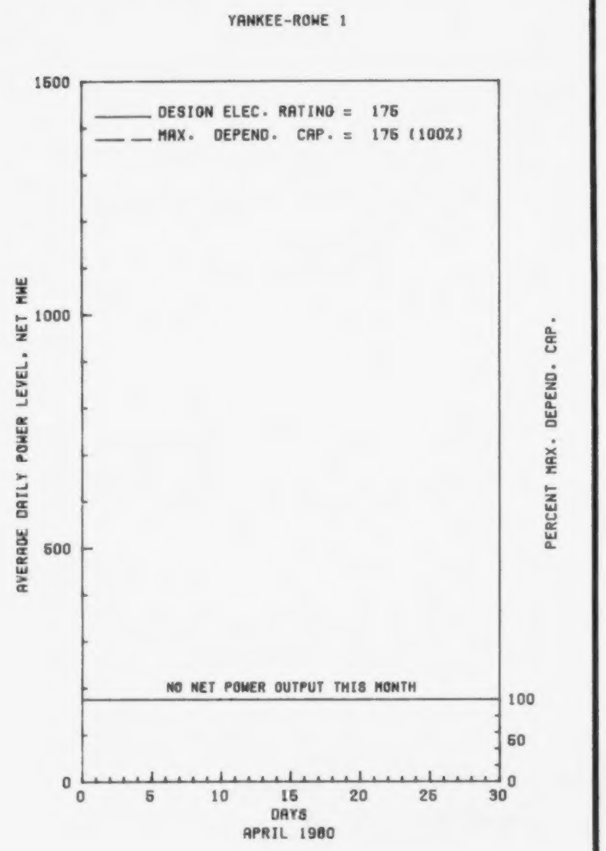
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-009/03L	03/07/80	04/04/80	SI time delay setpoint drift

REVIEWED BY OMPA LOEB: T.C. Cintula **DATE:** 6.12.80

REVIEWED BY NRR DOR: L. Burger **DATE:** 6-16-80

YANKEE ROWE

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-029
2. Utility Contact: F.C. BEERS (617) 366-9011
3. Licensed Thermal Power (MWt): 600
4. Nameplate Rating (Gross MWe): 185 x 1.0 = 185
5. Design Electrical Rating (Net MWe): 175
6. Maximum Dependable Capacity (Gross MWe): 180
7. Maximum Dependable Capacity (Net MWe): 175
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

CAPACITY FACTORS CALCULATED WITH A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	719.0	2,903.0	170,540.0
12. Hours Reactor Was Critical	.0	445.8	138,810.4
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	.0	439.7	134,467.1
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	253,769	72,002,091
17. Gross Electrical Energy (MWH)	0	80,635	22,152,788
18. Net Electrical Energy (MWH)	0	75,749	20,741,893
19. Unit Service Factor	.0	15.1	78.8
20. Unit Availability Factor	.0	15.1	78.8
21. Unit Capacity Factor (MDC Net)	.0	14.9	71.8
22. Unit Capacity Factor (DER Net)	.0	14.9	71.8
23. Unit Forced Outage Rate	100.0	83.1	3.1

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 08/15/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
1	01/19/80	F	719.0	A	4				SHUTDOWN FOR TURBINE ROTOR INVESTIGATION AND REPAIR CONTINUES.

SUMMARY

THE UNIT REMAINED SHUTDOWN THROUGHOUT THE REPORT PERIOD FOR TURBINE DISC REPAIR.

LEGEND

[1] Type:
F-Forced
S-Scheduled

[2] Reason:
A-Equipment Failure
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error
H-Other

[3] Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Continuations
5-Load Reduction
9-Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

ZION 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Illinois/Lake 40 miles N of Chicago
2.	Electric Reliability Council	Mid-America Interpool Network
3.	Type of Reactor	PWR
4.	Power Level (MWT)	3250
5.	Design Electrical Rating (MWE Net)	1040
6.	Date of Initial Criticality	June 19, 1973
7.	Date of Commercial Operation	December 31, 1973
8.	Condenser Cooling Method	Straight Through
9.	Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10.	Licensee	Commonwealth Edison Co.
11.	Corporate Address	P. O. Box 767 Chicago, Illinois
12.	Corporate Contact	D. L. Peoples
13.	Architect/Engineer	Sargent & Lundy
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Various

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	W. S. LITTLE
19.	IE Resident Inspector	J. E. Kohler (312) 746-2313
20.	Licensing Project Manager	E. Reeves
21.	Docket Number	50-295
22.	License Number/Date of Issue	DPR-39 issued 4/6/73
23.	Public Document Room	Zion-Benton Public Library 2600 Emmans Avenue Zion, Illinois 60099

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

Unit 1 operating routinely.

*LAST IE SITE INSPECTION DATE: February 6 and 27-28, 1980

* INSPECTION REPORT NO: 80-02

SUMMARY

INSPECTION SUMMARY

* Investigation on February 6 and 27-28, (80-02): Special, unannounced investigation into an allegation that a licensed reactor operator was under the influence of marijuana on January 17, 1980, while on duty in the control room of the Zion Generating Station, Unit 1. The investigation involved eight investigative hours. The allegation was not substantiated. No items of noncompliance with NRC requirements were identified during the course of the investigation.

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-10/ 03L-0	02/19/80	03/17/80	During normal operation the component cooling liquid rad monitor read high and alarmed intermittently.
80-11/ 03L-0	02/24/80	03/17/80	The Unit 1 Passive Gas Failure Rad Monitor, failed low and could not be reset.
80-12/ 03L-0	03/12/80	04/11/80	The D.C. control circuit breaker tripped.
80-13/ 03L-0	03/14/80	04/14/80	Surveillance of aux bldg. area monitor ORT-AR09, the IM deparment found that moniter read low by a factor greater than five.

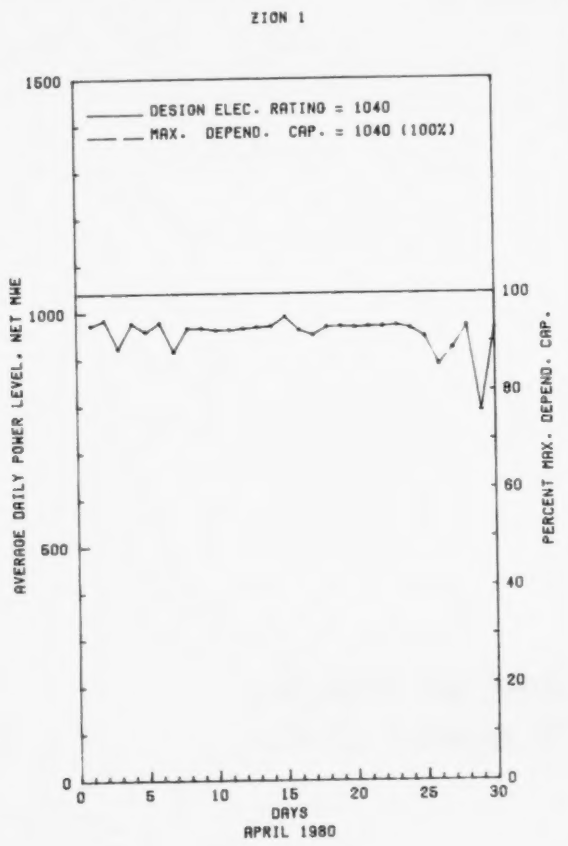
==

REVIEWED BY OMPA LOEB: T.C. Cintula **DATE:** 6·12·80

REVIEWED BY NRR DOR: Ed A. Reeves **DATE:** 6-13-80

ZION 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-295
2. Utility Contact: J. COOK (312) 746-2084
3. Licensed Thermal Power (MWt): 3250
4. Nameplate Rating (Gross MWe): 1220 x 0.9 = 1098
5. Design Electrical Rating (Net MWe): 1040
6. Maximum Dependable Capacity (Gross MWe): 1085
7. Maximum Dependable Capacity (Net MWe): 1040
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	719.0	2,903.0	55,511.0
12. Hours Reactor Was Critical	719.0	1,779.7	38,887.4
13. Reactor Reserve Shutdown Hours	.0	.0	2,621.8
14. Hours Generator On-Line	719.0	1,687.9	37,775.2
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	2,301,520	5,016,115	104,399,116
17. Gross Electrical Energy (MWH)	720,150	1,570,670	33,802,720
18. Net Electrical Energy (MWH)	687,240	1,476,922	31,961,168
19. Unit Service Factor	100.0	58.1	68.0
20. Unit Availability Factor	100.0	58.1	68.0
21. Unit Capacity Factor (MDC Net)	91.9	48.9	55.4
22. Unit Capacity Factor (DER Net)	91.9	48.9	55.4
23. Unit Forced Outage Rate	.0	41.9	15.0

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
8	04/26/80	F	.0	A	5				POWER REDUCED FOR THE PURPOSE OF RECONNECTING THE LEVEL SENSORS ON BERGEN-PATTERSON STEAM/GENERATOR EXPANSION TANK.

SUMMARY

OPERATION WAS AT NEAR FULL POWER DURING THE REPORT PERIOD, WITH A LOAD REDUCTION ON 04/26/80 TO RECONNECT LEVEL SENSORS ON A STEAM GENERATOR SNUBBER EXPANSION TANK.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

ZION 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Illinois/Lake 40 miles N of Chicago
2.	Electric Reliability Council	Mid-America Interpool Network
3.	Type of Reactor	PWR
4.	Licenses Power Level (MWT)	3250
5.	Design Electrical Rating (MWE Net)	1040
6.	Date of Initial Criticality	December 24, 1973
7.	Date of Commercial Operation	September 17, 1974
8.	Condenser Cooling Method	Straight Through
9.	Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10.	Licensee	Commonwealth Edison Co.
11.	Corporate Address	P. O. Box 767 Chicago, Illinois
12.	Corporate Contact	D. L. Peoples, Manager of Nuclear Licensing
13.	Architect/Engineer	Sergent & Lundy
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Various

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	W. S. LITTLE
19.	IE Resident Inspector	J. E. Kohler (312) 746-2313
20.	Licensing Project Manager	E. Reeves
21.	Docket Number	50-304
22.	License Number/Date of Issue	DPR-48, 11/14/73
23.	Public Document Room	Zion-Benton Public Library 2600 Emmans Avenue Zion, Illinois 60099

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

```
    I N S P E C T I O N   S T A T U S

    OTHER ITEMS OF INTEREST

    PLANT STATUS

*   Unit coasting down - routine refueling outage scheduled to begin
    May 2, 1980.

 LAST IE SITE INSPECTION DATE:  January 3 through February 6, 1980

        INSPECTION REPORT NO:  80-01

    SUMMARY

    INSPECTION SUMMARY

*   No inspections conducted.
```

REPORTS RECEIVED FROM LICENSEE

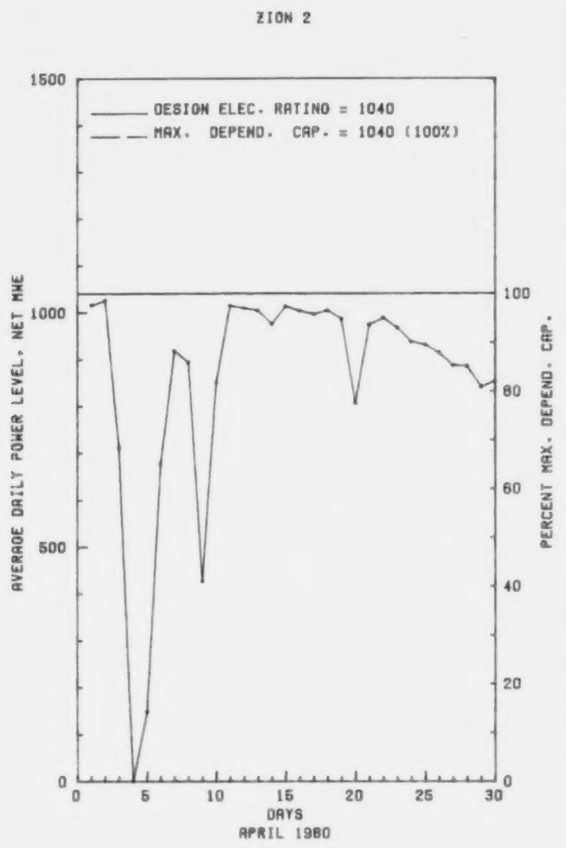
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-12/ 03L-0	02/24/80	03/25/80	Increasing power to 80%, control bank D rod position indicators were slightly greater than 12 steps above bank demand position.
80-13/ 03L-0	02/27/80	03/28/80	S/G pressure comparator 2PC-536A-B had shifted high by 1.5% (limit 1%).
80-14/ 03L-0	03/03/80	04/02/80	S/G pressure xmtr 2 PI-516 had shifted low by 2.0% (limit .5%).

REVIEWED BY OMPA LOEB: T. C. Cintula DATE: 6.12.80

REVIEWED BY NRR DOR: Ed L. Reeves DATE: 6-13-80

ZION 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-304
2. Utility Contact: J. COOK (312) 746-2084
3. Licensed Thermal Power (MWt): 3250
4. Nameplate Rating (Gross MWe): 1220 x 0.9 = 1098
5. Design Electrical Rating (Net MWe): 1040
6. Maximum Dependable Capacity (Gross MWe): 1085
7. Maximum Dependable Capacity (Net MWe): 1040
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	719.0	2,903.0	49,224.0
12. Hours Reactor Was Critical	686.2	2,410.8	36,014.6
13. Reactor Reserve Shutdown Hours	.0	.0	226.1
14. Hours Generator On-Line	673.2	2,381.3	35,195.2
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,976,715	7,314,221	99,949,539
17. Gross Electrical Energy (MWH)	646,855	2,395,725	32,116,945
18. Net Electrical Energy (MWH)	614,898	2,279,910	30,481,368
19. Unit Service Factor	93.6	82.0	71.5
20. Unit Availability Factor	93.6	82.0	71.5
21. Unit Capacity Factor (MDC Net)	82.2	75.5	59.5
22. Unit Capacity Factor (DER Net)	82.2	75.5	59.5
23. Unit Forced Outage Rate	6.4	18.0	17.7

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUELING - MAY 2, 1980 - APPROX. EIGHT WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
6	04/03/80	F	33.0	H	3				REACTOR TRIP DUE TO LIGHTENING.
7	04/05/80	F	3.7	A	3				REACTOR TRIP DUE TO 2A STEAM/GENERATOR LO-LO LEVEL.
8	04/05/80	F	9.1	A	3				REACTOR TRIP DUE TO 2A STEAM/GENERATOR LO-LO LEVEL.
9	04/09/80	F	.0	A	5				UNIT RAMPED DOWN DUE TO 2A-0 DIESEL/GENERATOR INOPERABILITY.
10	04/20/80	F	.0	A	5				UNIT RAMPED DOWN TO 50% TO ADD OIL TO 2A RCP; AND TO DO TURBINE STOP VALVE TESTING.
11	04/28/80	F	.0	A	5				POWER REDUCED TO 50% FOR IDENTIFYING REACTOR COOLANT SYSTEM LEAKAGE.

SUMMARY

OPERATION WAS ROUTINE DURING THE REPORT PERIOD, EXCEPT FOR REACTOR TRIPS ON 04/03/80 DUE TO LIGHTNING AND TWICE ON 04/05/80 DUE TO STEAM GENERATOR LO-LO LEVEL. LOAD REDUCTIONS OCCURRED ON 04/09/80 DUE TO PROBLEMS WITH TWO DIESEL GENERATORS, ON 04/20/80, TO ADD OIL TO A REACTOR COOLANT PUMP, AND ON 04/28/80 TO INVESTIGATE REACTOR COOLANT SYSTEM LEAKAGE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

SECTION 3 – APPENDIX

STATUS OF SPENT FUEL STORAGE CAPABILITY

DATA AS OF 4-30-80

Facility	Core Size [a] (No. of Assemblies)	Present Authorized Storage Pool Capacity (Fuel Assemblies)	No. of Assemblies Stored	Remaining Capacity (No. of Assemblies)	Remaining Capacity If Pending Request Approved (No. of Assemblies)	Scheduled Date For Next Refueling	Will Fill Present [b] Authorized Capacity
I. Pressurized Water Reactors							
ARKANSAS 1	177	590	176	414		12-80	1988
ARKANSAS 2	177	486	58	428		03-81	1989
BEAVER VALLEY 1	157	833	0	833		N/S	1995
CALVERT CLIFFS 1	217	1056(c)	364(c)	692(c)		10-80	1984
CALVERT CLIFFS 2	217					01-81	
COOK 1	193	500	193	307	1857	05-80	1985
COOK 2	193					N/S	
CRYSTAL RIVER 3	177	256	112	144	1105	N/S	1982
DAVIS-BESSE 1	177	735	0	735		N/S	1989
FARLEY 1	157	675	0	675		N/S	1990
FORT CALHOUN 1	133	483	197	286		03-81	1985
GINNA	121	595	156	439		N/S	1987
HADDAM NECK	157	1168	340	828		05-80	1994
INDIAN POINT 1	0	288	160	128		N/S	
INDIAN POINT 2	193	982	200	282		N/S	1984
INDIAN POINT 3	193	837	64	773		N/S	1989
KEWAUNEE	121	990	120	870		05-80	1999
MAINE YANKEE	217	953	433	520		N/S	1986
MILLSTONE 2	217	667	144	523		07-80	1987
NORTH ANNA 1	157	400	0	400	966	N/S	1984
NORTH ANNA 2(d)	157	400	0	400		N/S	
OCONEE 1	177	1224(l)	545	(k)	755	02-81	1980
OCONEE 2	177					N/S	
OCONEE 3	177					05-80	
PALISADES	204	798	273	525		N/S	1985
POINT BEACH 1	121	1502(c)	276(c)	1226(c)		11-80	1995
POINT BEACH 2	121					03-81	
PRAIRIE ISLAND 1	121	687(c)	280(c)	407(c)		07-80	1984
PRAIRIE ISLAND 2	121					02-81	
RANCHO SECO 1	177	579	151	428		07-81	1987
ROBINSON 2	157	272	106	166(e)		06-80	1980(g)
SALEM 1	193	264	40	224	1130	09-80	1982
SAN ONOFRE 1	157	216	97	119		N/S	1985
ST LUCIE 1	217	728	60	668		N/S	1990
SURRY 1	157	1044(c)	1963(c)	608(c)		03-80	1983
SURRY 2	157					09-80	
THREE MILE ISLAND 1	177	752	208	544		N/S	1989
THREE MILE ISLAND 2	177	442	0	442		N/S	1986
TROJAN	193	651	64	587		04-80	1988
TURKEY POINT 3	157	621	346(c)	275(c)		N/S	1987
TURKEY POINT 4	157					N/S	
YANKEE-ROWE 1	76	391	149	242	572	N/S	1985
ZION 1	193	868(c)	436(c)	432(c)	1676	01-81	1983
ZION 2	193					05-80	

Facility	Core Size [a] (No. of Assemblies)	Present Authorized Storage Pool Capacity (Fuel Assemblies)	No. of Assemblies Stored	Remaining Capacity (No. of Assemblies)	Remaining Capacity If Pending Request Approved (No. of Assemblies)	Scheduled Date For Next Refueling	Will Fill Present [b] Authorized Capacity
II. Boiling Water Reactors							
BIG ROCK POINT 1	84	193	68	105	353	10-80	1983
BROWNS FERRY 1	764	3471	576	2795		04-81	1985
BROWNS FERRY 2	764	1080	400	680		09-80	1985
BROWNS FERRY 3	764	1528	352	1176		10-80	1985
BRUNSWICK 1	560	(f)	(k)			05-80	1986
BRUNSWICK 2	560	(f)	91(PWR)+132(BWR)			N/S	1986
COOPER STATION	548	2366	284	2082		N/S	1995
DRESDEN 1	464	672	221	451		N/S	1986
DRESDEN 2	724	2840(c)	1069(c)	1771(c)	6491(c)	N/S	1985
DRESDEN 3	724					N/S	
DUANE ARNOLD	368	2050	364	1686		04-81	1998
FITZPATRICK	560	760	268	492	1752	05-80	1981
HATCH 1	560	840	260	580		N/S	1988
HATCH 2	560	1120	0	1120		N/S	
HUMBOLDT BAY	172	487	251	236		N/S	
LA CROSSE	72	134	113	21	327	08-80	1980
MILLSTONE 1	580	2184	776	1408		05-80	1991
MONTICELLO	484	2237	828	1409		03-80	1991
NINE MILE POINT 1	532	1984	660	1324	2349	N/S	1990
OYSTER CREEK 1	560	1800	780	1020		N/S	1987
PEACH BOTTOM 2	764	2816	910	1906		N/S	1990
PEACH BOTTOM 3	764	2816	712	2104		03-81	1991
PILGRIM 1	580	2320	764	1556		09-81	1990
QUAD CITIES 1	724	1460	596	864		09-80	1986
QUAD CITIES 2	724	1460	492	968		N/S	1985
VERMONT YANKEE 1	368	2000	894	1106		N/S	1988
III. Independent Spent Fuel Storage Installations (h)							
Morris Operations		750 MTU (j)	315	385 MTU (j)	1490 MTU (j)		
NFS(i)		250 MTU	170 MTU	80 MTU			

(a) At each refueling outage approximately 1/3 of a PWR core and 1/4 of a BWR core is off-loaded.
(b) Some of these dates have been adjusted by staff assumptions.
(c) This is the total for both units.
(d) Plant not in commercial operation.
(e) Some spent fuel stored at Brunswick.
(f) Authorized a total 2088 BWR and 304 PWR assemblies for both pools.
(g) Robinson 2 assemblies being shipped to Brunswick for storage.
(h) Capacity is in metric tons of uranium; 1 MTU = 2 PWR assemblies or 5 BWR assemblies.
(i) No longer accepting spent fuel.
(j) Racked for 700 MTU.
(k) Reserved
(l) This is the station total.

N/S = Not Scheduled

REACTOR-YEARS OF EXPERIENCE

AS OF 4/01/80

The total of Reactor years of experience is as the sum of all calendar days for each unit, from the date that electricity was first generated until a final shutdown date or the status date, whichever comes first, divided by 365.25 days/year.

If a date is unknown, the first day of the first month of operation is substituted. Units which have not yet generated electricity but which are licensed are listed but not included in the computation.

(INFORMATION INCLUDES BOTH LICENSED AND NON-LICENSED ELECTRICAL UNITS).

LICENSED OPERATING ELECTRICAL PRODUCING UNITS

UNIT NAME	FIRST ELEC. GENERATION	YEARS EXPERIENCE
ARKANSAS 1	08/01/74	5.75
ARKANSAS 2	12/26/78	1.18
BEAVER VALLEY 1	06/14/76	3.88
BIG ROCK POINT 1	12/08/62	17.40
BROWNS FERRY 1	10/15/73	6.54
BROWNS FERRY 2	08/28/74	5.67
BROWNS FERRY 3	09/12/76	3.63
BRUNSWICK 1	12/04/76	3.41
BRUNSWICK 2	04/29/75	5.01
CALVERT CLIFFS 1	01/03/75	5.33
CALVERT CLIFFS 2	12/07/76	3.40
COOK 1	02/10/75	5.22
COOK 2	03/22/78	2.11
COOPER STATION	05/10/74	5.98
CRYSTAL RIVER 3	01/30/77	3.25
DAVIS-BESSE 1	08/28/77	2.67
DRESDEN 1	04/15/60	20.04
DRESDEN 2	04/13/70	10.05
DRESDEN 3	07/22/71	8.78
DUANE ARNOLD	05/19/74	5.95
FARLEY 1	08/18/77	2.70
FITZPATRICK	02/01/75	5.25
FORT CALHOUN 1	08/25/73	6.68
FORT ST VRAIN	12/11/76	3.39
GINNA	12/02/69	10.41
HADDAM NECK	08/07/67	12.73
HATCH 1	11/11/74	5.47
HATCH 2	09/22/78	1.61
INDIAN POINT 2	06/26/73	6.85
INDIAN POINT 3	04/27/76	4.01
KEWAUNEE	04/08/74	6.06
LA CROSSE	04/26/68	12.01
MAINE YANKEE	11/08/72	7.48
MILLSTONE 1	11/29/70	9.42
MILLSTONE 2	11/09/75	4.43
MONTICELLO	03/05/71	9.16
NINE MILE POINT 1	11/09/69	10.48
NORTH ANNA 1	04/17/78	2.04
OCONEE 1	05/06/73	6.99
OCONEE 2	12/05/73	6.40
OCONEE 3	09/01/74	5.66
OYSTER CREEK 1	09/23/69	10.60
PALISADES	12/31/71	8.33
PEACH BOTTOM 2	02/18/74	6.20
PEACH BOTTOM 3	09/01/74	5.66
PILGRIM 1	07/19/72	7.78
POINT BEACH 1	11/06/70	9.48
POINT BEACH 2	08/02/72	7.75
PRAIRIE ISLAND 1	12/04/73	6.41
PRAIRIE ISLAND 2	12/21/74	5.36
QUAD CITIES 1	04/12/72	8.05
QUAD CITIES 2	05/23/72	7.94
RANCHO SECO 1	10/13/74	5.55
ROBINSON 2	09/26/70	9.60
SALEM 1	12/25/76	3.33

LICENSED OPERATING ELECTRICAL PRODUCING UNITS - CONT.

UNIT NAME	FIRST ELEC. GENERATION	YEARS EXPERIENCE
SAN ONOFRE 1	07/16/67	12.79
ST LUCIE 1	05/07/76	3.98
SURRY 1	07/04/72	7.82
SURRY 2	03/10/73	7.14
THREE MILE ISLAND 1	06/19/74	5.87
TROJAN	12/23/75	4.36
TURKEY POINT 3	11/02/72	7.49
TURKEY POINT 4	06/21/73	6.86
VERMONT YANKEE 1	09/20/72	7.61
YANKEE-ROWE 1	11/10/60	19.47
ZION 1	06/28/73	6.84
ZION 2	12/26/73	6.35
TOTAL		463.19 YEARS

PERMANENTLY OR INDEFINITELY SHUTDOWN UNITS

UNIT NAME	FIRST ELEC. GENERATION	SHUTDOWN OR ELECTRICAL CESSATION DATE	YEARS EXPERIENCE
BONUS	08/14/64	06/01/68	3.80
CVTR	12/18/63	01/01/67	3.04
ELK RIVER	08/24/63	02/01/68	4.44
FERMI 1	08/05/66	11/29/72	6.32
HALLAM	05/29/63	09/01/64	1.26
HUMBOLDT BAY	04/18/63	07/02/76	13.21
INDIAN POINT 1	09/16/62	10/31/74	12.12
PATHFINDER	07/25/66	10/01/67	1.19
PEACH BOTTOM 1	01/27/67	11/01/74	7.76
PIQUA	11/04/63	01/01/66	2.16
THREE MILE ISLAND 2	04/21/78	03/28/79	.93
TOTAL			56.23 YEARS

NON–POWER REACTORS IN THE U.S.

1. RESEARCH REACTORS

State	Location	Licensee	Reactor Type	Docket	License Number	Date Issued	Authorized Power Level (KW)
Alabama	1. Tuskegee	Tuskegee Institute	AGN 201 No. 102	50-406	R-122	08-30-74	.0001
Arizona	2. Tucson	U. of Arizona	TRIGA Mk. I	50-113	R-52	12-05-58	100
California	3. Los Angeles	U. of California	Argonaut	50-142	R-71	10-03-60	100
	4. Berkeley	U. of California	TRIGA Mark III	50-224	R-101	08-10-66	1000
	5. Irvine	U. of California	TRIGA Mark I	50-326	R-116	11-24-69	250
	6. Santa Barbara	U. of California	L-77	50-433	R-124	12-03-74	.01
	7. Canoga Park	Rockwell International Corporation	L-77	50-94	R-40	05-17-58	.01
	8. Hawthorne	Northrop Corporate Laboratories	TRIGA Mark F	50-187	R-90	03-04-63	1000
	9. San Luis Obispo	California State Polytechnic College	AGN 201 No. 100	50-394	R-121	05-16-73	.0001
	10. Alameda County	General Electric Co.	NTR	50-73	R-33	10-31-57	100
	11. San Diego	General Atomic Co.	TRIGA Mark I	50-89	R-38	05-03-58	250
	12. San Diego	General Atomic Co.	TRIGA Mark F	50-163	R-67	07-01-60	1500
	13. San Ramon	Aerotest Operations, Inc.	TRIGA (Industrial)	50-228	R-98	07-02-65	250
Colorado	14. Jefferson County	U.S. Geological Survey. Dept. of Interior	TRIGA Mark I	50-274	R-113	02-24-69	1000
Delaware	15. Newark	U. of Delaware	AGN-201 No. 113	50-98	R-43	07-03-58	.0001
District of Columbia	16. –	Catholic U.	AGN-201 No. 101	50-77	R-31	11-15-67	.0001
Florida	17. Gainesville	U. of Florida	Argonaut	50-83	R-56	05-21-59	100
Georgia	18. Atlanta	Georgia Institute of Technology	AGN-201 No. 104	50-276	R-111	04-19-68	.0001
	19. Atlanta	Georgia Institute of Technology	Heavy Water	50-160	R-97	12-29-64	5000
Idaho	20. Pocatello	Idaho State U.	AGN-201 No. 103	50-284	R-110	10-11-67	.0001
Illinois	21. Urbana	U. of Illinois	Advanced TRIGA	50-151	R-115	07-22-69	1500
	22. Urbana	U. of Illinois	LOPRA	50-356	R-117	12-27-71	10
	23. Zion	Westinghouse Electric Corporation	NTR	50-87	R-119	01-28-72	10
Indiana	24. West Lafayette	Purdue U.	Lockheed	50-182	R-87	08-16-62	10

RESEARCH REACTORS – CONT.

State	Location	Licensee	Reactor Type	Docket	License Number	Date Issued	Authorized Power Level (KW)
Iowa	25. Ames	Iowa State U.	UTR-10	50-116	R-59	10-16-59	10
Kansas	26. Lawrence	U. of Kansas	Lockheed	50-148	R-78	06-23-61	250
	27. Manhattan	Kansas State U.	TRIGA	50-188	R-88	10-16-62	250
Maryland	28. College Park	U. of Maryland	TRIGA	50-166	R-70	10-14-60	250
	29. Bethesda	Armed Forces Radiobiology Research Institute	TRIGA	50-170	R-84	06-26-62	1000
Massachusetts	30. Lowell	U. of Lowell	GE	50-223	R-125	12-24-74	1000
	31. Cambridge	Massachusetts Institute of Technology	Heavy Water Reflected	50-20	R-37	06-09-58	5000
	32. Worcester	Worcester Polytechnic Institute	GE	50-134	R-61	12-16-59	10
Michigan	33. Ann Arbor	U. of Michigan	Pool	50- 2	R-28	09-13-57	2000
	34. Lansing	Michigan State U.	TRIGA Mark I	50-294	R-114	03-21-69	250
	35. Midland	Dow Chemical Co.	TRIGA Mark I	50-264	R-108	07-03-67	100
Missouri	36. Rolla	U. of Missouri	Pool	50-123	R-79	11-21-61	200
	37. Columbia	U. of Missouri	Tank	50-186	R-103	10-11-66	10,000
Nebraska	38. Omaha	V.A. Hospital	TRIGA	50-131	R-57	06-26-59	18
New Mexico	39. Albuquerque	U. of New Mexico	AGN-201M No. 112	50-252	R-102	09-17-66	.005
New York	40. Buffalo	State U. of New York	PULSTAR	50-57	R-77	03-24-61	2000
	41. Sterling Forest	Union Carbide Corp.	Pool	50-54	R-81	09-07-61	5000
	42. New York City	Columbia U.	TRIGA Mark II	50-208	R-128	04-14-77	250
	43. Ithaca	Cornell U.	ZPR	50-97	R-89	12-11-62	.1
	44. Ithaca	Cornell U.	TRIGA Mk. II	50-157	R-80	01-11-62	100
	45. Riverdale	Manhattan College	Tank	50-199	R-94	03-24-64	.0001
North Carolina	46. Raleigh	North Carolina State U.	PULSTAR	50-297	R-120	08-25-72	1000
Ohio	47. Columbus	Ohio State U.	Pool	50-150	R-75	02-24-61	10
Oklahoma	48. Norman	U. of Oklahoma	AGN-211 No. 102	50-112	R-53	12-29-58	.015
Oregon	49. Corvallis	Oregon State U.	AGN-201 No. 114	50-106	R-51	11-13-58	.0001
	50. Corvallis	Oregon State U.	TRIGA Mark II	50-243	R-106	03-07-67	1000
	51. Portland	Reed Institute	TRIGA Mk. I	50-288	R-112	07-02-68	250
Pennsylvania	52. University Park	Pennsylvania State U.	TRIGA Mk. III	50-5	R-2	07-08-55	1000

NON–POWER REACTORS IN THE U.S.

RESEARCH REACTORS – CONT.

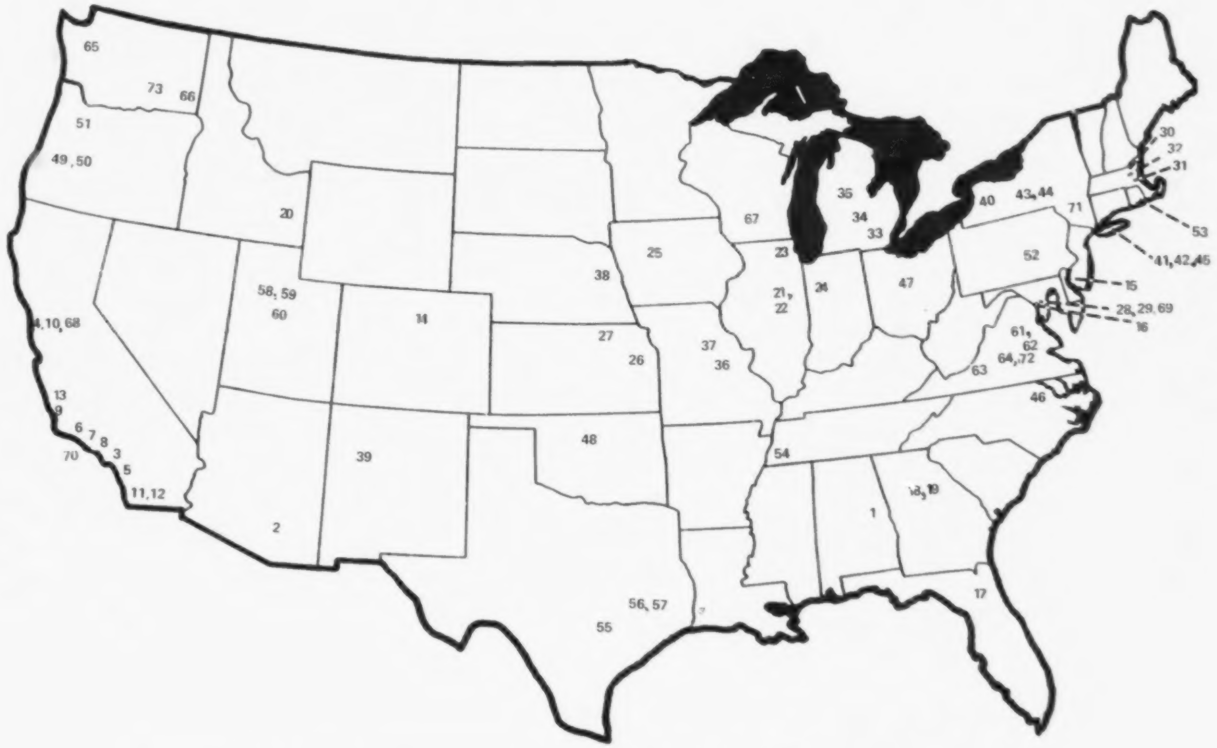
State	Location	Licensee	Reactor Type	Docket	License Number	Date Issued	Authorized Power Level (KW)
Rhode Island	53. Ft. Kearney, Narragansett	Rhode Island & Providence Plantations AEC	GE Pool	50-193	R-95	07-21-64	2000
Tennessee	54. Shelby County	Memphis State U.	AGN-201 No. 108	50-538	R-127	12-10-76	.0001
Texas	55. Austin	U. of Texas	TRIGA Mk. I	50-192	R-92	08-02-63	250
	56. College Station	Texas A&M U.	TRIGA	50-128	R-83	12-07-61	1000
	57. College Station	Texas A&M U.	AGN- 201M No. 106	50-59	R-23	08-26-57	.005
Utah	58. Salt Lake City	U. of Utah	AGN-201M No. 107	50-72	R-25	09-12-57	.005
	59. Salt Lake City	U. of Utah	TRIGA Mark I	50-407	R-126	09-30-75	100
	60. Provo	Brigham Young U.	L-77	50-262	R-109	09-07-67	.01
Virgina	61. Charlottesville	U. of Virgina	Pool	50-62	R-66	06-27-60	2000
	62. Charlottesville	U. of Virginia	Cavalier	50-396	R-123	09-24-74	.1
	63. Blacksburg	Virginia Polytechnic Institute	UTR-10	50-124	R-62	12-18-59	100
	64. Lynchburg	Babcock & Wilcox Co.	LPR	50-99	R-47	09-05-58	1000
Washington	65. Seattle	U. of Washington	Argonaut	50-139	R-73	03-31-61	100
	66. Pullman	Washington State U.	TRIGA	50-27	R-76	03-06-61	1000
Wisconsin	67. Madison	U. of Wisconsin	TRIGA	50-156	R-74	11-23-60	1000

2. EXPERIMENTAL AND TEST REACTORS

State	Location	Licensee	Reactor Type	Docket	License Number	Date Issued	Authorized Power Level (MWt)
California	68. Alameda Co.	General Electric Co.	GETR	50-70	TR-1	01-07-59	50
Maryland	69. Gaithersburg	National Bureau of Standards	TEST	50-184	TR-5	11-13-67 POL 06-30-70 OL	10

3. CRITICAL EXPERIMENT FACILITIES

State	Location	Licensee	Docket	License Number	Date Issued	Experiments
California	70. Ventura Co.	Rockwell International Corp. (Possesion Only)	50-147	CX-17	10-03-60	AETR Experiments (Terminal Requested).
New York	71. Schenectady	Rensselaer Polytechnic Institute	50-225	CX-22	07-03-64	
Virginia	72. Lynchburg	Babcock & Wilcox Co.	50-13	CX-10	10-22-58	NMSR Experiments
Washington	73. Richland	Battelle Memorial Institute	50-360	CX-26	11-29-71	Plutonium Recycle Critical Facility



Source: NRC Facilities License Application Record (FLAR) 06-30-77.

ABBREVIATIONS

AGN	Aerojet General Nuclear	These reactors were constructed by Aeroject at San Ramon, California, and then transferred to the current location.
ZPR	Zero Power Reactor	
LPR	Low Power Reactor	

DISTRIBUTION LIST–NRC

INTERNAL DISTRIBUTION

Office of the Commissioners	5
Atomic Safety and Licensing Appeal Panel	3
Advisory Committee on Reactor Safeguards	6
Office of Inspector and Auditor	1
Office of Policy Evaluation	1
Office of the General Counsel	1
Office of Public Affairs	21
Office of Congressional Affairs	3
Office of the Executive Director for Operations	7
Office of Administration	2
Office of the Executive Legal Director	2
Office of the Controller	2
Office for Analysis and Evaluation of Operational Data	2
Office of International Programs	3
Office of State Programs	1
Office of Management and Program Analysis	14
Office of Standards Development	1
Office of Nuclear Material Safety and Safeguards	2
– Division of Fuel Cycle and Material Safety	3
– Division of Safeguards	2
Office of Nuclear Reactor Regulation	5
– Division of Project Management	6
– Division of Operating Reactors	105
– Division of Systems Safety	4
– Division of Site Safety and Environmental Analysis	18
Office of Nuclear Regulatory Research	4
Office of Inspection and Enforcement	7
– Region I	11
– Region II	10
– Region III	10
– Region IV	10
– Region V	7
	279

EXTERNAL DISTRIBUTION

SPECIAL REQUESTS

Air Force	1
Congress	10
Department of Energy	32
Government Accounting Office	1
Environmental Protection Agency	1
Electric Power Research Institute	1
Argonne National Laboratory	3
Federal Power Commission	1
Bureau of Mines	1
Department of Agriculture	1
Department of Commerce	1
Department of the Interior	1
	54

OTHER

GPO Depository	440
GPO Stores	75
National Technical Information Service	25
Subscriptions (NTIS)	250
Colleges and Libraries (including Public Document Rooms)	128
Utilities and Other Requests	227
	1145

15/46

Nuclear Regulatory Commission

OPERATING UNITS STATUS REPORT

DATA AS OF 5-31-80

NUREG 0020
VOL. 4 NO. 6
JUNE 1980

NEWARK PUBLIC LIBRARY
AUG 5 1980
U. S. GOVT. PUBS.
DEPOSITORY 376

LICENSED OPERATING REACTORS

Y3.N88:

Available from
GPO Sales Program
Division of Technical Information and Document Control
U. S. Nuclear Regulatory Commission
Washington, D.C. 20555
and
National Technical Information Service
Springfield, Virginia 22161
703-326-8543
A Year's Subscription, 12 Issues, for this Publication is
$120.00 North American Continent ($240 00 Other).
Multiple Subscriptions Entered at the Same Time to the Same Address
Cost $90.00 North American Continent ($180.00 Other) for each Additional Subscription.

Single copies are $12.50 North American Continent ($25.00 Other).

This document is provided free of charge to those who participate in the data collection program, and to the Nuclear Regulatory Commission's local public document rooms.

Issued by the U.S. Nuclear Regulatory Commission

PUBLISHED: JULY 11, 1980

Nuclear Regulatory Commission

OPERATING UNITS STATUS REPORT

DATA AS OF 5-31-80

NUREG 0020
VOL. 4 NO. 6
JUNE 1980

LICENSED OPERATING REACTORS

AUTHORIZATION AND CLEARANCE

THE U.S. NUCLEAR REGULATORY COMMISSION'S OFFICE OF MANAGEMENT AND PROGRAM ANALYSIS PUBLISHES THIS MONTHLY STATUS REPORT "AS PART OF THE REPORTING REQUIREMENTS IN SECTION 50.36 OF 10 CFR PART 50 UNDER GAO CLEARANCE NUMBER B-180225, WITH AN EXPIRATION DATE OF SEPTEMBER 30, 1981," AS STATED IN THE OCTOBER 3, 1978 LETTER FROM JOHN M. LOVELADY, ASSISTANT DIRECTOR, GENERAL GOVERNMENT DIVISION, U. S. GENERAL ACCOUNTING OFFICE, TO J.M. FELTON, DIRECTOR, DIVISION OF RULES AND RECORDS, U.S. NUCLEAR REGULATORY COMMISSION.

STATEMENT OF PURPOSE

The U. S. Nuclear Regulatory Commission's monthly OPERATING UNITS STATUS REPORT – LICENSED OPERATING REACTORS, provides data on the operation of nuclear units as timely and accurately as possible. This information is collected by the Office of Management and Program Analysis from the Headquarters Staff of NRC's Office of Inspection and Enforcement, from NRC's Regional Offices, and from utilities. Since all of the data concerning operation of the units is provided by the utility operators less than two weeks after the end of the month, necessary corrections to published information are shown on the ERRATA page.

This report is divided into three sections: the first contains monthly highlights and statistics for commercial operating units, and errata from previously reported data; the second is a compilation of detailed information on each unit, provided by NRC Regional Offices, IE Headquarters and the Utilities; and the third section is an appendix for miscellaneous information such as spent fuel storage capability, reactor-years of experience and non-power reactors in the United States.

The percentage computations, Items 19 through 24 in Section 2, the vendor capacity factors on page 1-4, and actual vs. potential energy production on Page 1-2 are computed from actual power production from the period of consideration. The percentages listed in power generation on Page 1-2 are computed as an arithmetical average. The factors for the life-span of each unit (the "Cumulative" column) are reported by the utility and are not entirely re-computed by NRC. Utility power production data is checked for consistency with previously submitted statistics.

In some cases, the numbers in the "Cumulative" column may have been totaled from a date that is different from the date of commercial operation. In these cases, the column is noted with an explanation of the nature of the change on each individual plant page in Section 2.

It is hoped this status report proves informative and helpful to all agencies and individuals interested in analyzing trends in the nuclear industry which might have safety implications, or in maintaining an awareness of the U. S. energy situation as a whole.

PHILIP A. ROSS
REVIEW COORDINATOR
EARL HILL
TECHNICAL ASSISTANT
LEONARD TOMSON
TECHNICAL ASSISTANT

TABLE OF CONTENTS

GLOSSARY ii
INDEX TO LICENSED UNITS iii

SECTION 1 – CURRENT DATA SUMMARIES

MONTHLY HIGHLIGHTS OF COMMERCIAL NUCLEAR POWER UNITS 1-2

- Licensed Power Reactors 1-2
- Power Generation 1-2
- Actual vs. Potential Energy Production 1-2
- Outage Data 1-2
- Derated Units 1-2
- Shutdowns - Number and Reasons 1-3
- Unit Availability, Capacity, and Forced Outage Rate 1-3
- Refueling Schedule 1-3

AVERAGE CAPACITY FACTORS BY VENDOR 1-4

AVERAGE DAILY POWER LEVEL FOR ALL COMMERCIAL OPERATING UNITS 1-5

ERRATA 1-7

SECTION 2 – OPERATING POWER REACTORS

ARKANSAS 1 THROUGH ZION 2 PAGES A–2 TO Z–5

- Facility Data
- Inspection and Enforcement Status
- Reports Received from Licensee
- Operating Status
- Average Daily Power Level (MWe)
- Unit Shutdowns/Reductions

SECTION 3 – APPENDIX

STATUS OF SPENT FUEL STORAGE CAPABILITY 3-2
REACTOR-YEARS OF EXPERIENCE 3-3
NON-POWER REACTORS IN THE U.S. 3-4

GLOSSARY

AVERAGE DAILY POWER LEVEL (MWe)	The net electrical energy generated during the day (measured from 0001 to 2400 hours inclusive) in megawatts hours divided by 24 hours.
LICENSED THERMAL POWER (MWt)	The maximum thermal power of the reactor authorized by the NRC, expressed in megawatts.
DATE OF COMMERCIAL OPERATION	Date unit was declared by utility owner to be available for the regular production of electricity; usually related to satisfactory completion of qualification tests as specified in the purchase contract and to accounting policies and practices of utility.
DESIGN ELECTRICAL RATING (DER) (Net MWe)	The nominal net electrical output of the unit specified by the utility and used for the purpose of plant design.
FORCED OUTAGE	An outage required to be initiated no later than the weekend following discovery of an off-normal condition.
FORCED OUTAGE HOURS	The clock hours during the report period that a unit is unavailable due to forced outages.
GROSS ELECTRICAL ENERGY GENERATED (MWH)	Electrical output of the unit during the report period as measured at the output terminals of the turbine generator, in megawatts hours.
GROSS HOURS	The clock hours from the beginning of a specified situation until its end. For outage durations, the clock hours during which the unit is not in power production.
GROSS THERMAL ENERGY GENERATED (MWH)	The thermal energy produced by the unit during the report period as measured or computed by the licensee in megawatt hours.
HOURS GENERATOR ON-LINE	Also, "Unit Service Hours." The total clock hours in the report period during which the unit operated with breakers closed to the station bus. These hours added to the total outage hours experienced by the unit during the report period, shall equal the hours in the report period.
HOURS IN REPORTING PERIOD	For units in power ascension at the end of the period, the gross hours from the beginning of the period or the first electrical production, whichever comes last, to the end of the period. For units in commercial operation at the end of the period, the gross hours from the beginning of the period or of commercial operation, whichever comes last, to the end of the period or decommissioning, whichever comes first.
HOURS REACTOR CRITICAL	The total clock hours in the report period during which the reactor sustained a controlled chain reaction.
MAXIMUM DEPENDABLE CAPACITY (GROSS) (MDC Gross) (Gross MWe)	Dependable main-unit gross capacity, winter or summer, whichever is smaller. The dependable capacity varies because the unit efficiency varies during the year due to cooling water temperature variations. It is the gross electrical output as measured at the output terminals of the turbine generator during the most restrictive seasonal conditions (usually summer).
MAXIMUM DEPENDABLE CAPACITY (NET) (MDC Net) (Net MWe)	Maximum Dependable Capacity (Gross) less the normal station service loads.
NAMEPLATE RATING (Gross MWe)	The nameplate power designation of the generator in megavolt amperes (MVA) times the nameplate power factor of the generator. NOTE: The nameplate rating of the generator may not be indicative of the maximum or dependable capacity, since some other item of equipment of a lesser rating (e.g. , turbine) may limit unit output.
NET ELECTRICAL ENERGY GENERATED	Gross electrical output of the unit measured at the output terminals of the turbine generator during the reporting period, minus the normal station service electrical energy utilization. If this quantity is less than zero, a negative number should be recorded.
OUTAGE	A situation in which no electrical production takes place.
OUTAGE DATE	As reported on Appendix D of Reg. Guide 1.16, the date of the start of the outage. If continued from a previous month, report the same outage date but change "Method of Shutting Down Reactor" to "4 (Other)" and add a note: "Continued from previous month."
OUTAGE DURATION	The total clock hours of the outage measured from the beginning of the report period or the outage, whichever comes last, to the end of the report period or the outage, whichever comes first.
OUTAGE NUMBER	A number unique to the outage assigned by the licensee. The same number is reported each month in which the outage is in progress. One format is "76-05" for the fifth outage to occur in 1976.
PERIOD HOURS	See "Hours in Reporting Period."
POWER REDUCTION	A reduction in the Average Daily Power Level of more than 20% from the previous day. All power reductions are defined as outage of zero hours durations for the purpose of computing unit service and availability factors, and forced outage rate.
REACTOR AVAILABLE HOURS	The total clock hours in the report period during which the reactor was critical or was capable of being made critical. (Reactor Reserve Shutdown Hours + Hours Reactor Critical.)
REACTOR AVAILABILITY FACTOR	$\frac{\text{Reactor Available Hours} \times 100}{\text{Period Hours}}$
REACTOR RESERVE SHUTDOWN	The cessation of criticality in the reactor for administrative or other similar reasons when operation could have been continued.
REACTOR RESERVE SHUTDOWN HOURS	The total clock hours in the report period during which the reactor is in reserve shutdown mode. NOTE: No credit is given for NRC imposed shutdowns.
REACTOR SERVICE FACTOR	$\frac{\text{Hours Reactor Critical} \times 100}{\text{Period Hours}}$
REPORT PERIOD	Usually, the preceding calendar month. Can also be the preceding calendar year, (Year-to-Date), or the life-span of a unit (cumulative).
RESTRICTED POWER LEVEL	Maximum net electrical generation to which the unit is restricted during the report period due to the state of equipment, external conditions, administrative reasons, or a direction by NRC.
SCHEDULED OUTAGE	Planned removal of a unit from service for refueling, inspection, training, or maintenance. Those outages which do not fit the definition of "Forced Outage" perforce are "Scheduled Outages."
STARTUP AND POWER ASCENSION TEST PHASE	Period following initial criticality during which the unit is tested at successively higher levels, culminating with operation at full power for a sustained period and completion of warranty runs. Following this phase, the utility generally considers the unit to be available for commercial operation.
UNIT	The set of equipment uniquely associated with the reactor, including turbine generators, and ancillary equipment, considered as a single electrical energy production facility.
UNIT AVAILABLE HOURS	The total clock hours in the report period during which the unit operated on-line or was capable of such operation. (Unit Reserve Shutdown Hours + Hours Generator On-Line.)
UNIT AVAILABILITY FACTOR*	$\frac{\text{Unit Available Hours} \times 100}{\text{Period Hours}}$
UNIT CAPACITY FACTORS	
– Using Licensed Thermal Power	$\frac{\text{Gross Thermal Energy Generated} \times 100}{\text{Period Hours} \times \text{Lic. Thermal Power}}$
– Using Nameplate Rating	$\frac{\text{Gross Electrical Energy Generated} \times 100}{\text{Period Hours} \times \text{Nameplate Rating}}$
– Using DER	$\frac{\text{Net Electrical Energy Generated} \times 100}{\text{Period Hours} \times \text{DER}}$
– Using MDC Gross	$\frac{\text{Gross Electrical Energy Generated} \times 100}{\text{Period Hours} \times \text{MDC Gross}}$
– Using MDC Net	$\frac{\text{Net Electrical Energy Generated} \times 100}{\text{Period Hours} \times \text{MDC Net}}$

NOTE: if MDC Gross and/or MDC Net have not been determined, the DER is substituted for this quantity for Unit Capacity Factor calculations.

UNIT FORCED OUTAGE RATE	$\frac{\text{Forced Outage Hours}}{\text{Unit Service Hours} + \text{Forced Outage Hours}}$
UNIT RESERVE SHUTDOWN	The removal of the unit from on-line operation for economic or other similar reasons when operation could have been continued.
UNIT RESERVE SHUTDOWN HOURS	The total clock hours in the report period during which the unit was in reserve shutdown mode.
UNIT SERVICE FACTOR	$\frac{\text{Unit Service Hours} \times 100}{\text{Period Hours}}$
UNIT SERVICE HOURS	See "Hours Generator On-Line."

INDEX TO LICENSED UNITS

ARKANSAS 1 A-3
ARKANSAS 2 A-5
BEAVER VALLEY 1 B-3
BIG ROCK POINT 1 B-5
BROWNS FERRY 1 B-7
BROWNS FERRY 2 B-9
BROWNS FERRY 3 B-11
BRUNSWICK 1 B-13
BRUNSWICK 2 B-15
CALVERT CLIFFS 1 C-3
CALVERT CLIFFS 2 C-5
COOK 1 C-7
COOK 2 C-9
COOPER STATION C-11
CRYSTAL RIVER 3 C-13
DAVIS-BESSE 1 D-3
DRESDEN 1 D-5
DRESDEN 2 D-7
DRESDEN 3 D-9
DUANE ARNOLD D-11
FARLEY 1 F-3
FITZPATRICK F-5
FORT CALHOUN 1 F-7
FORT ST VRAIN F-9
GINNA G-3
HADDAM NECK H-3
HATCH 1 H-5
HATCH 2 H-7
HUMBOLDT BAY H-9
INDIAN POINT 1 I-3
INDIAN POINT 2 I-5
INDIAN POINT 3 I-7
KEWAUNEE K-3
LA CROSSE L-3
MAINE YANKEE M-3
MILLSTONE 1 M-5
MILLSTONE 2 M-7
MONTICELLO M-9
NINE MILE POINT 1 N-3
NORTH ANNA 1 N-5
OCONEE 1 O-3
OCONEE 2 O-5
OCONEE 3 O-7
OYSTER CREEK 1 O-9
PALISADES P-3
PEACH BOTTOM 2 P-5
PEACH BOTTOM 3 P-7
PILGRIM 1 P-9
POINT BEACH 1 P-11
POINT BEACH 2 P-13
PRAIRIE ISLAND 1 P-15
PRAIRIE ISLAND 2 P-17
QUAD CITIES 1 Q-3
QUAD CITIES 2 Q-5
RANCHO SECO 1 R-3
ROBINSON 2 R-5
SALEM 1 S-3
SAN ONOFRE 1 S-5
ST LUCIE 1 S-7
SURRY 1 S-9
SURRY 2 S-11
THREE MILE ISLAND 1 T-3
THREE MILE ISLAND 2 T-5
TROJAN T-7
TURKEY POINT 3 T-9
TURKEY POINT 4 T-11
VERMONT YANKEE 1 V-3
YANKEE-ROWE 1 Y-3
ZION 1 Z-3
ZION 2 Z-5

SECTION 1 – CURRENT DATA SUMMARIES

MONTHLY HIGHLIGHTS OF COMMERCIAL NUCLEAR POWER UNITS

FOR MONTH OF

MAY 1980

LICENSED POWER REACTORS:

NUMBER		CAPACITY[1] MWe (Net)
67	IN COMMERCIAL OPERATION	49,010
67	LICENSED TO OPERATE	49,010
3[3]	LICENSED FOR FUEL LOADING AND LOW POWER TESTING	

POWER GENERATION	REPORT MONTH	PREVIOUS MONTH	YEAR-TO-DATE
1. GROSS ELECTRICAL (MWHE)	19,313,241	19,135,853	99,656,412
2. NET ELECTRICAL (MWHE)	18,384,016	18,263,263	94,979,772
3. AVG. UNIT SERVICE FACTOR (%)	57.0	56.2	59.2
4. AVERAGE UNIT AVAILABILITY FACTOR (%)	57.0	55.5	59.4
5. AVERAGE UNIT CAPACITY FACTOR (MDC) (%)	48.8	49.2	52.6
6. AVG. UNIT CAPACITY FACTOR (DER) (%)	47.6	48.1	51.3
7. FORCED OUTAGE RATE (%)	10.6	11.5	10.7

NOTES

[1] BASED UPON MAXIMUM DEPENDABLE CAPACITY; DESIGN ELECTRIC RATING USED IF MDC NOT DETERMINED.

[2] EXCLUDES INDIAN POINT 1 (DER = 265), HUMBOLDT BAY (DER = 65), AND THREE MILE ISLAND 2 (DER = 906) WHICH HAVE OPERATING LICENSES BUT ARE SHUTDOWN INDEFINITELY.

[3] SEQUOYAH 1 (DER = 1148) WAS LICENSED ON FEBRUARY 29, 1980
NORTH ANNA 2 (DER = 907) WAS LICENSED ON APRIL 11, 1980
SALEM 2 (DER = 1115) WAS LICENSED ON APRIL 18, 1980

ACTUAL VS. POTENTIAL ENERGY PRODUCTION

		% OF POTENTIAL PRODUCTION
POTENTIAL ENERGY PRODUCTION IN THIS PERIOD BY UNITS IN COMMERCIAL OPERATION (Using Maximum Dependable Capacity Net)	36,479,064 MWHe	
ENERGY ACTUALLY PRODUCED DURING THIS REPORT PERIOD =	18,384,016 MWHe NET	50.4
ENERGY NOT PRODUCED DURING THIS REPORT PERIOD DUE TO SCHEDULED OUTAGES (NET) =	11,656,707 MWHe	32.0
ENERGY NOT PRODUCED DURING THIS REPORT PERIOD DUE TO FORCED OUTAGES (NET) =	3,699,580 MWHe	10.1
ENERGY NOT PRODUCED DURING THIS REPORT PERIOD FOR OTHER REASONS (NET)	2,739,761 MWHe	7.5
TOTAL	36,479,064 MWHe =	100%
ENERGY NOT PRODUCED THIS REPORT PERIOD DUE TO NRC IMPOSED SHUTDOWNS AND RESTRICTIONS =	1,844,453 MWHe	5.1
NUMBER OF UNITS WITH POWER RESTRICTIONS IMPOSED BY NRC		

OUTAGE DATA

	NUMBER	HOURS	PERCENT OF CLOCK TIME	MWHE LOST PRODUCTION
1. FORCED OUTAGES DURING MONTH	61	5,153.9	10.3	3,699,580
2. SCHEDULED OUTAGES DURING THE MONTH	42	16,271.6	32.6	11,656,707
3. TOTAL	103	21,425.5	43.0	15,356,287

MWHE LOST PRODUCTION = Down time x maximum dependable capacity net

DERATED UNITS

UNIT NAME	MAXIMUM DEPENDABLE CAPACITY (MWe - NET)	POWER LIMIT (MWe - NET)
BIG ROCK POINT 1	65	65
DUANE ARNOLD	515	250
†FORT ST. VRAIN	330	231
POINT BEACH 1	495	390
ROBINSON 2	2300 MWt	2200 MWt

†Restricted by NRC.

MONTHLY HIGHLIGHTS OF COMMERCIAL NUCLEAR POWER UNITS

FOR MONTH OF

MAY 1980

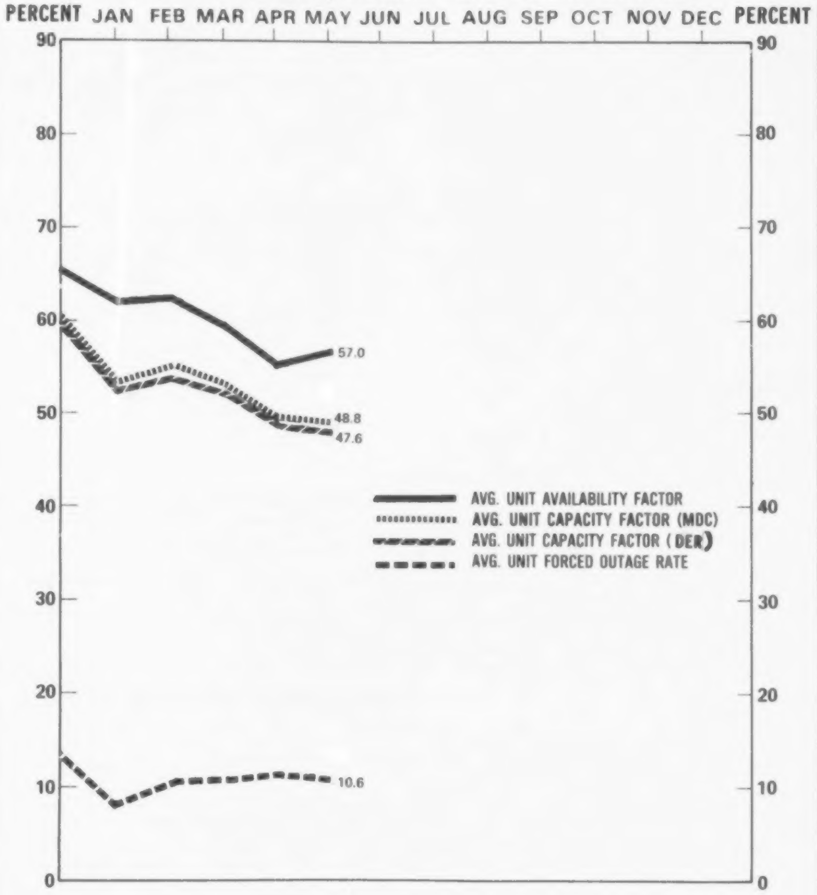
REASONS FOR SHUTDOWN

REASON	NUMBER	TOTAL HOURS LOST
A - Equipment Failure	42	3,516.2
B - Maintenance or Test	14	1,803.9
C - Refueling	22	11,992.0
D - NRC Restriction	5	2,317.3
E - Operator Training and License Examination	0	0
F - Administrative	2	173.0
G - Operational Error	8	84.9
H - Other	10	1,538.2
TOTAL	103	21,425.5

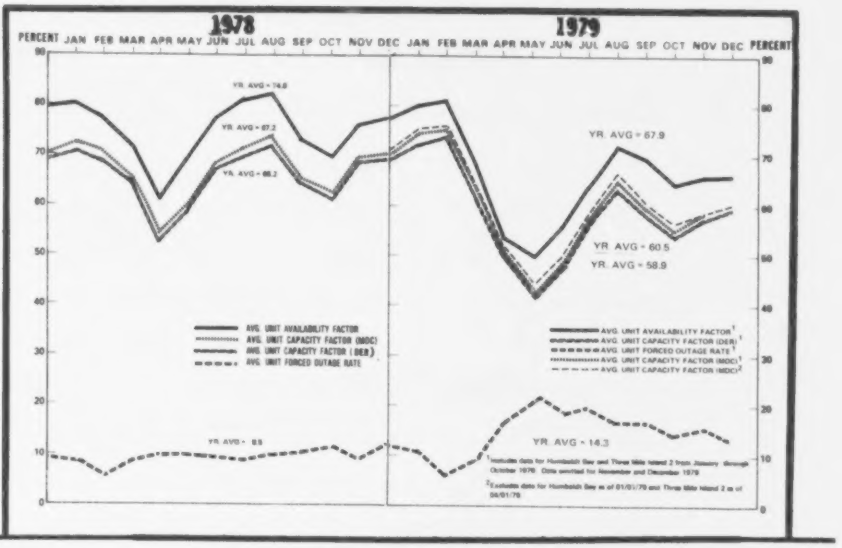
SHUTDOWNS GREATER THAN 72 HOURS EACH

UNIT NAME	REASON	UNIT NAME	REASON
ARKANSAS 1	A	NORTH ANNA 1	A
ARKANSAS 2	F	OCONEE 2	B
BEAVER VALLEY 1	H	OYSTER CREEK 1	C
BRUNSWICK 1	C	PALISADES	H
BRUNSWICK 2	C	PEACH BOTTOM 2	C
CALVERT CLIFFS 1	A	PEACH BOTTOM 3	A
COOPER STATION	C	PILGRIM 1	C
CRYSTAL RIVER 3	C	POINT BEACH 2	C
DAVIS-BESSE 1	C	PRAIRIE ISLAND 2	B
DRESDEN 1	H	RANCHO SECO 1	C
DRESDEN 2	A	SAN ONOFRE 1	C
DUANE ARNOLD	A	ST LUCIE 1	B,C
FITZPATRICK	C	SURRY 1	D
FORT CALHOUN 1	C	SURRY 2	D
GINNA	C	THREE MILE ISLAND 1	D
HADDAM NECK	C	TROJAN	C
HATCH 1	A	TURKEY POINT 4	B
HATCH 2	A	YANKEE-ROWE 1	A
KEWAUNEE	C	ZION 2	C
MILLSTONE 2	D		
MONTICELLO	A		

UNIT AVAILABILITY, CAPACITY AND FORCED OUTAGE



UNIT AVAILABILITY, CAPACITY AND FORCED OUTAGE



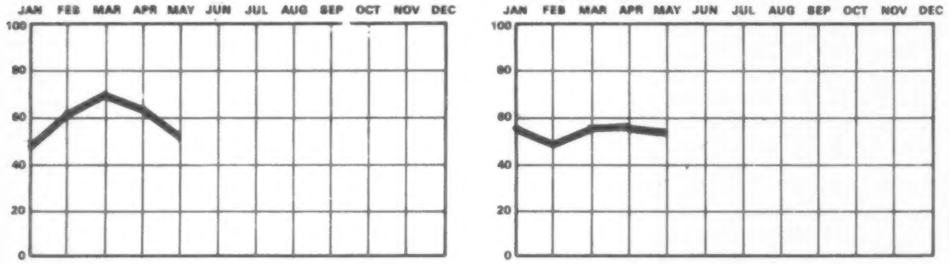
REFUELING SCHEDULE

Please refer to the table on page 3-2 for the scheduled refueling date for each nuclear powerplant in this publication.

VENDOR AVERAGE CAPACITY FACTORS

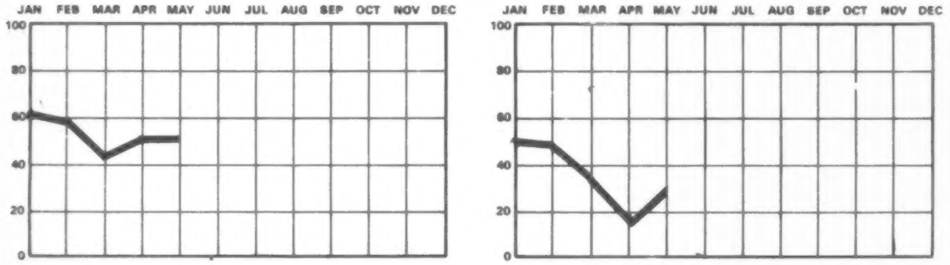
WESTINGHOUSE

COMBUSTION ENGINEERING



GENERAL ELECTRIC

BABCOCK & WILCOX



NOTE: THIS DISPLAY OF AVERAGE CAPACITY FACTORS PROVIDES A GENERAL PERFORMANCE COMPARISON OF PLANTS SUPPLIED BY THE FOUR NUCLEAR STEAM SUPPLY SYSTEM VENDORS. ONE MUST BE CAREFUL WHEN DRAWING CONCLUSIONS REGARDING THE REASONS FOR THE PERFORMANCE LEVELS INDICATED, SINCE PLANT PERFORMANCE MAY BE AFFECTED BY UNSPECIFIED FACTORS SUCH AS: (1) VARIOUS PLANT DESIGNS AND MODELS ARE INCLUDED FOR EACH VENDOR; (2) TURBINE/GENERATORS AND (3) DIFFERENT ARCHITECT/ENGINEERS ARE ALSO INVOLVED.

MAY 1980

OPERATIONS

GE BWR's	CFMDC
BROWNS FERRY 1	83.8
BROWNS FERRY 2	87.6
BROWNS FERRY 3	88.4
BRUNSWICK 1	51.7
BRUNSWICK 2	.0
COOPER STATION	.0
DRESDEN 1	.0
DRESDEN 2	65.9
DRESDEN 3	68.6
DUANE ARNOLD	44.4
FITZPATRICK	12.9
HATCH 1	66.8
HATCH 2	62.1
MILLSTONE 1	85.3
MONTICELLO	50.3
NINE MILE POINT 1	97.2
OYSTER CREEK 1	.0
PEACH BOTTOM 2	.0
PEACH BOTTOM 3	30.9
PILGRIM 1	20.8
QUAD CITIES 1	74.0
QUAD CITIES 2	77.1
VERMONT YANKEE 1	99.8

23 UNITS

W PWR's	
BEAVER VALLEY 1	.0
COOK 1	95.2
COOK 2	97.8
FARLEY 1	97.6
GINNA	20.9
HADDAM NECK	6.1
INDIAN POINT 2	89.9
INDIAN POINT 3	70.1
KEWAUNEE	28.1
NORTH ANNA 1	64.1
POINT BEACH 1	75.5
POINT BEACH 2	46.4
PRAIRIE ISLAND 1	90.8
PRAIRIE ISLAND 2	83.0
ROBINSON 2	61.6
SALEM 1	95.0
SAN ONOFRE 1	.0
SURRY 1	55.4
SURRY 2	.0
TROJAN	.0
TURKEY POINT 3	87.3
TURKEY POINT 4	.0
YANKEE-ROWE 1	.0
ZION 1	92.6
ZION 2	2.1

25 UNITS

CE PWR's	CFMDC
ARKANSAS 2	78.2
CALVERT CLIFFS 1	83.4
CALVERT CLIFFS 2	99.2
FORT CALHOUN 1	0.0
MAINE YANKEE	76.3
MILLSTONE 2	24.1
PALISADES	9.8
ST LUCIE 1	45.3

8 UNITS

B&W PWR's	CFMDC
ARKANSAS 1	9.2
CRYSTAL RIVER 3	.0
DAVIS-BESSE 1	.0
OCONEE 1	73.4
OCONEE 2	.0
OCONEE 3	94.1
RANCHO SECO 1	47.8
THREE MILE ISLAND 1	.0

8 UNITS

	GE BWR's	W PWR's	CE PWR's	B&W PWR's	All PWR's
NET ELECTRICAL PRODUCTION	6,736,816	7,472,678	2,564,471	1,437,112	11,474,253
MDC NET	17,327	18,428	[illegible]	6,737	31,255
CFMDC	52.3	54.6	56.6	28.5	49.3

Capacity factor in this page, denoted as CFMDC, is a function of the net maximum dependable capacity. See the corresponding definition in the Glossary. The vendor averages are computed by the formula:

$$\frac{\text{Net Electrical Energy Produced by Vendor}}{\text{Potential Electrical Production By Vendor In This Month.}} \times 100\%$$

Units Excluded	Reason for Exclusion
Indian Point 1 Humboldt Bay Three Mile Island 2	Unit Shutdown Indefinitely
Big Rock Point	Not Representative of Typical BWRs
LaCrosse	Not Representative of Typical BWRs and Atypical Vendor
Fort St. Vrain	Atypical Vendor and Reactor Design

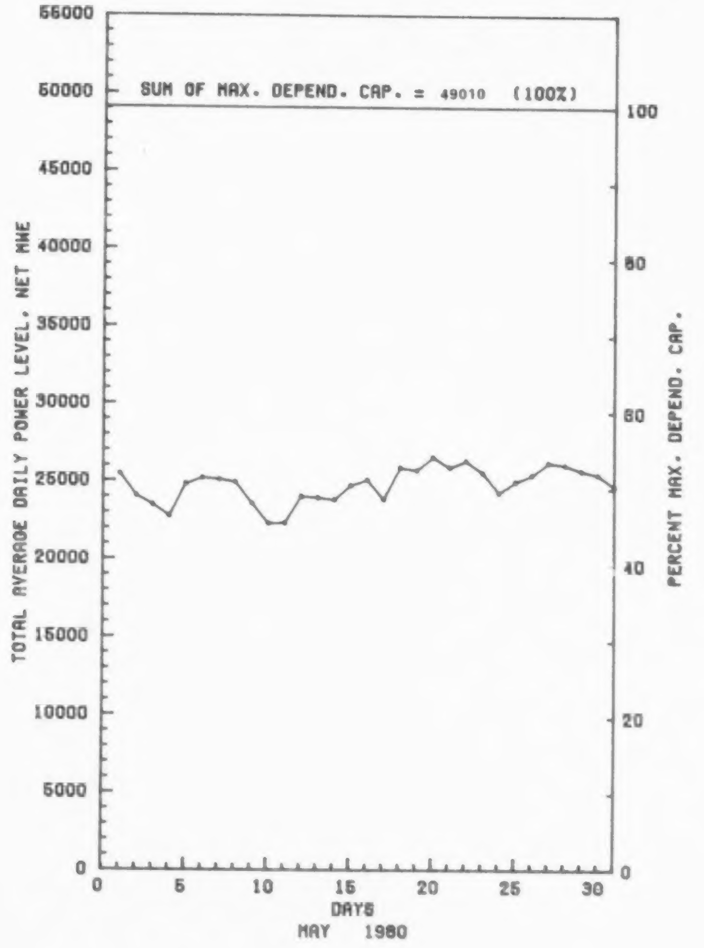
AVERAGE DAILY POWER LEVEL FOR ALL COMMERCIALLY OPERATING UNITS

This chart depicts the average daily power level for the units in commercial operation during the month.

The straight line on the graph labelled "SUM OF MAX. DEPEND. CAP." is plotted at the value shown by summing the separate maximum dependable capacities of the commercially operating units (in Net MWe). The plot shown below the line is calculated by summing the separate average daily power levels of the commercially operating units for each day of the month.

The scale on the left vertical axis runs in 1,000 MWe increments from 0 to 55,000 MWe (Net). The right vertical axis shows the percentage in 10% increments, up to 100% of the "SUM OF MAX. DEPEND. CAP."

It should be recognized that the 100% line would be obtainable only if all of the commercially operating units operated at 100% capacity, 24 hours per day, for the entire month. In other words, since any power generator must occasionally shut down to refuel and/or perform needed maintenance, and also since 100% capacity production is not always required by power demands, the 100% line is a theoretical goal and not a practical one.



best copy available

ERRATA

Corrections and Amendments to Previously-Reported Data

(NOTE: These changes are reflected in the data contained in the current report)

CORRECTIONS TO NUREG-0020 VOL. 4 NO. 5 MAY 1980

Page	Unit Name	Item	Correction
	NONE		

MULTIPLE - ISSUE CORRECTIONS

NONE

SECTION 2 – OPERATING POWER REACTORS

ARKANSAS 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Arkansas/Pope 6 miles NW of Russellville
2.	Electric Reliability Council	Southwest Power Pool
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2568
5.	Design Electrical Rating (MWE Net)	850
6.	Date of Initial Criticality	August 6, 1974
7.	Date of Commercial Operation	December 19, 1974
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Dardenelle Reservoir

Utility & Contractor Information

10.	Licensee	Arkansas Power and Light Company
11.	Corporate Address	P.O. Box 551 Little Rock, Arkansas 72203
12.	Corporate Contact	W. Cavanaugh III Vice Pres., Generation & Construction
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	B&W
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	IV
17.	IE Branch Chief	G. L. Madsen
18.	IE Section Chief	T. F. Westerman
19.	IE Resident Inspector	W. D. Johnson (501) 968-3290
20.	Licensing Project Manager	G. S. Vissing
21.	Docket Number	50-313
22.	License Number/Date of Issue	DPR-51, 5/21/74
23.	Public Document Room	Arkansas Polytechnic College Russellville, Arkansas 72801

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO 10 CFR PART 20.203(C), "HIGH RADIATIONAREAS," THE FOLLOWING ACCESSES TO POSTED HIGH RADIATION AREAS WERE FOUND TO BE UNLICKED AND OPEN WITH NO POSITIVE ACCESS CONTROL ON APR 2, 1980: 1. DOOR 137 TO THE "B" CHARGING PUMP (P36B) ROOM. ACCESSIBLE WHOLE BODY RADIATION WAS IN EXCESS OF 300 MILLIREMS PER HOUR. 2. ACCESS TO SEAL RETURN COOLER ROOM (EL. 369'). ACCESSIBLE WHOLE BODY RADIATION WAS IN EXCESS OF 350 MILLIREMS PER HR.
(8006 INFRACTION)

CONTRARY TO PROCEDURE 1053.01, CONTROL OF COMBUSTIBLES. TWO BUCKETS OF OIL WERE LEFT IN THE INTAKE STRUCTURE AND AN OXYGEN AND AN ACETYLENE BOTTLE WERE LEFT IN THE LOWER SOUTH PIPING PENETRATION ROOM.
(8006 INFRACTION)

CONTRARY TO THE RADIATION PROTECTION MANUAL, 1602.35, THE WASTE CONTROL OPERATOR ENTERED SEVERAL HIGH RADIATION AREAS WITHOUT A RADIATION SURVEY INSTRUMENT OR HEALTH PHYSICS COVERAGE.
(8006 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

A RCP seal failed on May 10, 1980. During current outage all four RCP seals will be removed, inspected, and rebuilt.

FACILITY ITEMS (PLANS AND PROCEDURES):

MANAGERIAL ITEMS

PLANT STATUS:

The unit is in a maintenance outage resulting from RCP seal failure on May 10, 1980.

LAST IE SITE INSPECTION DATE: March 22 - April 21, 1980

INSPECTION REPORT NO: 50-313/80-06

SUMMARY

INSPECTION SUMMARY

Inspection conducted during period of March 22 - April 21, 1980 (80-06): Routine, announced inspection including Emergency Planning Drills, Surveillance Observation, Follow-up on IE Circulars, Follow-up on Licensee Event Reports, Operational Safety Verification, and Organization and Administration. Within the six areas inspected, three items of noncompliance were identified (infraction - control of high radiation areas; infraction - control of combustibles; use of radiation survey instrument in high radiation areas.)

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-09 03L-0	3/28/80	4/30/80	Hydrogen purge supply fan failure
80-11 03L-0	4/1/80	4/28/80	Inoperable hangers
80-12 03L-0	4/6/80	5/2/80	EFW pump steam supply valve failures
80-13 03L-0	4/7/80	5/5/80	High pressure injection valve failure
80-14 03L-0	4/15/80	5/12/80	Testing fire and smoke detectors in reactor building
80-15 01T-0	5/10/80	5/19/80	Reactor coolant pump seal failure

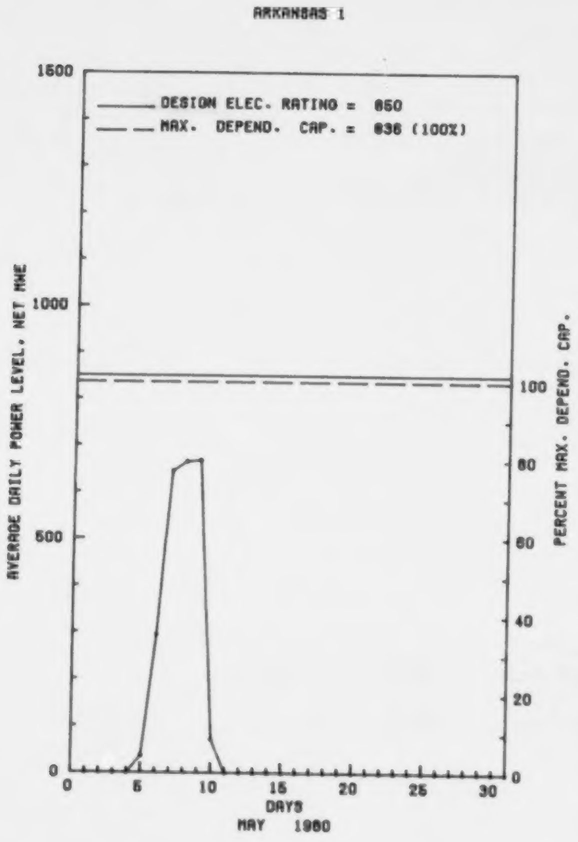
REVIEWED BY OMPA LOEB: PE Bobe **DATE:** 7/3/80

REVIEWED BY NRR DOR: **DATE:** 7/5/80

GPO 871-667

ARKANSAS 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-313
2. Utility Contact: L. S. Bramlett (501) 968-2519
3. Licensed Thermal Power (MWt): 2568
4. Nameplate Rating (Gross MWe): 1003 x 0.9 = 903
5. Design Electrical Rating (Net MWe): 850
6. Maximum Dependable Capacity (Gross MWe): 883
7. Maximum Dependable Capacity (Net MWe): 836
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	3,647.0	47,778.0
12. Hours Reactor Was Critical	128.7	1,710.4	31,848.6
13. Reactor Reserve Shutdown Hours	90.2	464.8	4,712.2
14. Hours Generator On-Line	109.3	1,671.6	31,142.1
15. Unit Reserve Shutdown Hours	.0	20.8	817.5
16. Gross Thermal Energy (MWH)	201,913	3,844,695	75,106,739
17. Gross Electrical Energy (MWH)	60,927	1,198,925	24,920,081
18. Net Electrical Energy (MWH)	57,078	1,140,352	23,774,388
19. Unit Service Factor	14.7	45.8	65.2
20. Unit Availability Factor	14.7	46.4	66.9
21. Unit Capacity Factor (MDC Net)	9.2	37.4	59.5
22. Unit Capacity Factor (DER Net)	9.0	36.8	58.5
23. Unit Forced Outage Rate	82.8	28.0	16.4

24. Shutdowns Scheduled Over Next 6 Months (Type,Date,and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 06/06/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-03	0[illegible]/19/80	S	105.4	F	4		ZZ	ZZZZZZ	THE OUTAGE WAS ADMINISTRATIVELY AUTHORIZED FOR ECONOMIC REASONS.
80-04	05/05/80	S	4.1	B	1		ZZ	ZZZZZZ	TURBINE OVERSPEED TRIP TEST.
80-05	05/10/80	F	525.2	A	1		CA	PUMPXX	"C" RCP SEAL FAILURE.

SUMMARY

THE UNIT BEGAN THE REPORTING PERIOD IN A COLD SHUTDOWN MODE DUE TO OUTAGE WORK. IT WAS BROUGHT BACK ON LINE ON 05/05/80. ON 05/10/80, IT WAS SHUTDOWN DUE TO COOLANT PUMP SEAL MALFUNCTION. THE UNIT REMAINED DOWN FOR THE REST OF THE MONTH FOR MAINTENANCE.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

ARKANSAS 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Arkansas/Pope 6 miles NW of Russellville
2.	Electric Reliability Council	Southwest Power Pool
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2815
5.	Design Electrical Rating (MWE Net)	912
6.	Date of Initial Criticality	12-5-78
7.	Date of Commercial Operation	3/26/80
8.	Condenser Cooling Method	Recirculating
9.	Condenser Cooling Water Source	Cooling Tower

Utility & Contractor Information

10.	Licensee	Arkansas Power & Light Company
11.	Corporate Address	P.O. Box 551 Little Rock, Arkansas 72203
12.	Corporate Contact	W. Cavanaugh III Vice Pres., Generation & Construction
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Combustion Engineering, Inc.
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	IV
17.	IE Branch Chief	G. L. Madsen
18.	IE Section Chief	T. F. Westerman
19.	IE Resident Inspector	W. D. Johnson (501) 968-3290
20.	Licensing Project Manager	R. E. Martin 492-8087
21.	Docket Number	50-368
22.	License Number/Date of Issue	NFP-6, 9/1/78
23.	Public Document Room	Arkansas Polytechnic College Russellville, Arkansas 72801

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO 10 CFR PART 20.203(C), "HIGH RADIATION AREAS," THE FOLLOWING DOORS TO POSTED HIGH RADIATION AREAS WERE FOUND TO BE UNLOCKED AND OPEN WITH NO POSITIVE ACCESS CONTROL" A. DOOR 226 TO THE 'B' CHARGING PUMP (P36B) ROOM ON APR 2, 1980. ACCESSIBLE WHOLE BODY RADIATION WAS IN EXCESS OF 1000 MILLIREMS PER HR. 2. DOOR 436 TO SPENT FUEL DEM'N ROOM ON APR 9, 1980. ACCESSIBLE WHOLE BODY RADIATION WAS IN EXCESS OF 60000 MILLIREM PER HR. 3. DOOR 217 TO ION EXCHANGE AREA ON APR 11, 1980. ACCESSIBLE WHOLE BODY RADIATION WAS IN EXCESS OF 100 MILLIREMS PER HR.
(8006 INFRACTION)

VALVES 2CV-5672-1 AND 2CV-5673-1 DID NOT HAVE SEISMIC SUPPORT BRACKETS INSTALLED AS REQUIRED BY ISOMETRIC DRAWINGS 2DCB-11-1 AND 2DCB-13-1.
(8006 INFRACTION)

CONTRARY TO PROCEDURE 1053.01, CONTROL OF COMBUSTIBLES. TWO SAFETY CANS OF OIL WERE LEFT IN THE CORRIDOR OUTSIDE THE EMERGENCY FEEDWATER PUMP ROOMS
(8006 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

Steam driven EFW pump (2P7A) has tripped due to overspeed on several occasions during the past month. The licensee has increased test frequency of the pump to maintain operability pending a permanent fix.

FACILITY ITEMS (PLANS AND PROCEDURES):

MANAGERIAL ITEMS:

PLANT STATUS:

The unit is operating at 90% power.

LAST IE INSPECTION DATE: March 22 - April 21, 1980

INSPECTION REPORT NO: 50-368/80-06 +

SUMMARY

INSPECTION SUMMARY

Inspection conducted during period of March 22 - April 21, 1980 (80-06): Routine, announced inspection including Emergency Planning Drills, Transient Test Witnessing, Follow-up on IE Circulars, Operational Safety Verification, Organization and Administration, Core Thermal Power, and Maintenance. Within the seven areas inspected, three items of noncompliance were identified (infraction - control of high radiation areas; infraction (control of combustibles; and infraction - missing seismic support brackets).

REPORTS RECEIVED FROM LICENSEE

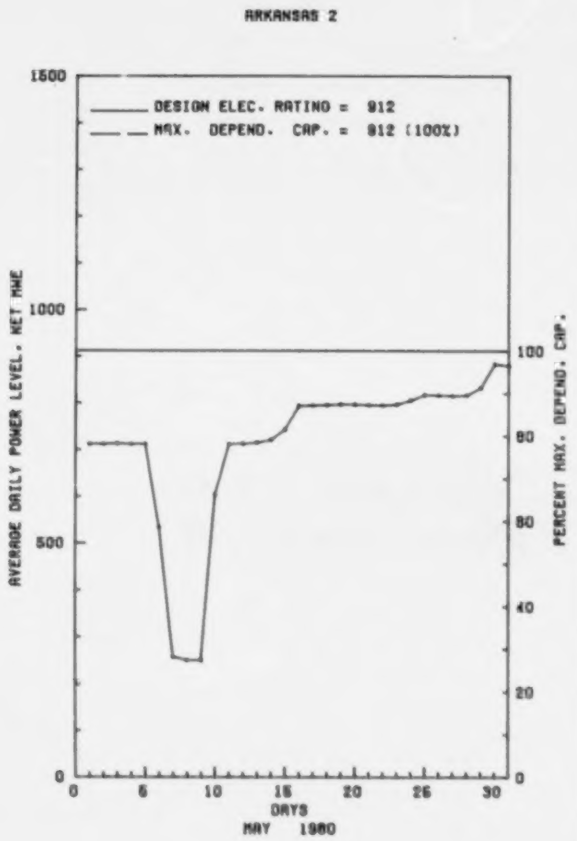
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-15 99X-0	1/29/80	5/13/80	Safety injection nozzle thermal cycle
80-18 03L-0	4/7/80	5/5/80	Loss of emergency feedwater pump suction
80-19 03L-0	4/10/80	5/8/80	Charging pump suction line break
80-20 03L-0	4/18/80	5/14/80	Pressurizer safety valve inoperable
80-21 04L-0	4/19/80	5/15/80	Condenser gas sample monitor inoperable
80-22 03L-0	4/24/80	5/20/80	Steam driven emergency feedwater pump failure
80-24 03L-0	4/8/80	5/20/80	Containment spray train inoperable

REVIEWED BY OMPA LOEB: L. [illegible] **DATE:** 7/3/80

REVIEWED BY NRR DOR: RS Martin **DATE:** 7/7/80

ARKANSAS 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-368
2. Utility Contact: Rex Pendergraft (501) 968-2519
3. Licensed Thermal Power (MWt): 2815
4. Nameplate Rating (Gross MWe): 959
5. Design Electrical Rating (Net MWe): 912
6. Maximum Dependable Capacity (Gross MWe): 912
7. Maximum Dependable Capacity (Net MWe): 912
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	1,607.0	1,607.0
12. Hours Reactor Was Critical	744.0	1,277.0	1,277.0
13. Reactor Reserve Shutdown Hours	.0	80.9	80.9
14. Hours Generator On-Line	744.0	1,250.6	1,250.6
15. Unit Reserve Shutdown Hours	.0	75.0	75.0
16. Gross Thermal Energy (MWH)	1,696,789	2,888,656	2,888,656
17. Gross Electrical Energy (MWH)	557,890	950,030	950,030
18. Net Electrical Energy (MWH)	530,284	903,265	903,265
19. Unit Service Factor	100.0	77.8	77.8
20. Unit Availability Factor	100.0	82.5	82.5
21. Unit Capacity Factor (MDC Net)	78.2	61.6	61.6
22. Unit Capacity Factor (DER Net)	78.2	61.6	61.6
23. Unit Forced Outage Rate	.0	22.2	22.2

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	05/08/80								REDUCED POWER FOR UNIT 1 STARTUP - REDUCED TRANSMISSION CAPABILITY.

SUMMARY

THE UNIT OPERATED AT NEAR FULL POWER FOR REPORTING PERIOD WITH A REDUCED POWER CAPABILITY FOR THE BEGINNING OF THE MONTH, DUE TO LACK OF TRANSMISSION CAPABILITY.

LEGEND

[1] Type:
F–Forced
S–Scheduled

[2] Reason:
A–Equipment Failure
B–Maintenance or Test
C–Refueling
D–Regulatory Restriction
E–Operator Training & License Examination
F–Administrative
G–Operational Error
H–Other

[3] Method:
1–Manual
2–Manual Scram
3–Automatic Scram
4–Continuations
5–Load Reduction
9–Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

NRC POWER RESTRICTIONS: NONE

BEAVER VALLEY 1

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Pennsylvania/Beaver 5 miles E of E. Liverpool, Ohio
2.	Electric Reliability Council	East Central Area Reliability Coordination Agreement
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2660
5.	Design Electrical Rating (MWE Net)	852
6.	Date of Initial Criticality	May 10, 1976
7.	Date of Commercial Operation	10/01/76 (50%) 04/30/77 (100%)
8.	Condenser Cooling Method	Cooling tower/Closed loop
9.	Condenser Cooling Water Source	Ohio River

Utility & Contractor Information

10.	Licensee	Duquesne Light Company
11.	Corporate Address	435 Sixth Avenue Pittsburgh, Pennsylvania 15219
12.	Corporate Contact	C. N. Dunn, Vice President
13.	Architect/Engineer	Stone & Webster
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Section Chief	E. C. MCCABE
19.	IE Resident Inspector	D. A. BECKMAN (412-643-9200/9201)
20.	Licensing Project Manager	W. J. Ross
21.	Docket Number	50-334
22.	License Number/Date of Issue	DPR-66, 1/30/76
23.	Public Document Room	B. F. Jones Memorial Library 633 Franklin Avenue Aliquippa, PA 15001

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

FACILITY ITEMS (PLANS AND PROCEDURES):

Refueling/modification outage commenced 11/30/79 and is scheduled to continue until 7/22/80.

PLANT STATUS:

In cold shutdown. Major modifications are underway on the following safety-related systems: Bergen-Patterson large bore snubbers; Grinnel small bore snubbers; containment spray pumps; refueling water storage tank; low head safety injection pumps; inside recirc spray pumps; pipe hangers; auxiliary feedwater pumps; low pressure turbine rotors; main transformer; and unit station service transformer.

LAST IE SITE INSPECTION DATE: 5/30/80

INSPECTION REPORT NO: 50-334/80-14

SUMMARY

INSPECTION SUMMARY

50-334/80-06 - Feb 11 - Mar 7: Routine inspections by the resident inspectors (100 hours) of: action on previous inspection findings; plant operations; refueling operations; licensee event review; review of physical protection/plant security; and review of a potential for containment hydrogen recombiner failures. One item of noncompliance was identified: (Failure to maintain fire penetration barrier integrity).

50-334/80-08 - Mar 24-28: Routine announced inspection (75 hours) by one region-based inspector and one headquarters-based Engineer of the Quality Assurance Program (QAP) implementation including: QA Program Review; QA/QC Administration; Design Change/Modification Controls; Offsite Support Staff (including Training); Document Control; Test and Measurement; and followup on previously identified items. No items of noncompliance were identified.

50-334/80-10 - Apr 8-9: Unannounced investigation (20 hours) by two NRC regional investigators of two allegations concerning weld rod control and work practices of an electrical subcontractor. None of the allegations could be substantiated. No items of noncompliance were identified.

50-334/80-11 - Apr 15-17: Routine, unannounced inspection by one region-based inspector (24 hours) of welding and nondestructive examination activities associated with the recirculation spray and low head safety injection system modification, and licensee action on previous inspection findings. No items of noncompliance were identified.

REPORTS RECEIVED FROM LICENSEE

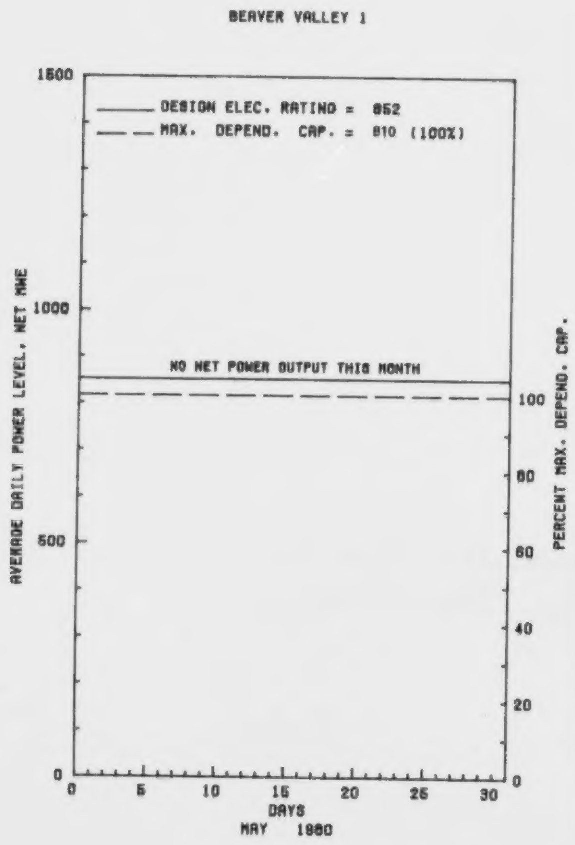
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-020/ 03L	03/29/80	04/29/80	Fuse holder fault in lo-lo VCT/RWST ckt caused VCT outlet to shut & RWST outlet to open as if vct level were low. Fuse holder replacement corrected problem.
80-022/ 01T	04/02/80	04/22/80	A loss of RHR flow occurred when operators were trying to increase flow in mode 5 w/the steam generators drained & the RCS at midspan level in the loops when the pumps bound
80-023/ 01T	04/11/80	04/23/80	Complete RHR flow loss during bypass valve closure to increase flow
80-025/ 01T	04/15/80	04/16/80	12 inch nozzle weld to RWST does not meet penetration requirements
80-029/ 03L	04/09/80	05/06/80	Containment isolation check valve SI-42 on accum fill line failed periodic test for operating differential pressure. Investigation of failure & repairs being accomplished

REVIEWED BY OMPA LOEB: T.C. Cintula DATE: 7.3.80

REVIEWED BY NRR DOR: W J Ross DATE: 7/7/80

BEAVER VALLEY 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-334
2. Utility Contact: DAVID R. TIMKO (412) 643-5108
3. Licensed Thermal Power (MWt): 2660
4. Nameplate Rating (Gross MWe): 1026 x 0.9 = 923
5. Design Electrical Rating (Net MWe): 852
6. Maximum Dependable Capacity (Gross MWe): 845
7. Maximum Dependable Capacity (Net MWe): 810
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	3,647.0	35,807.0
12. Hours Reactor Was Critical	.0	.0	13,774.7
13. Reactor Reserve Shutdown Hours	.0	.0	4,482.1
14. Hours Generator On-Line	.0	.0	13,105.0
15. Unit Reserve Shutdown Hours	.0	.0	0
16. Gross Thermal Energy (MWH)	0	0	26,974,253
17. Gross Electrical Energy (MWH)	0	0	8,277,940
18. Net Electrical Energy (MWH)	0	0	7,570,743
19. Unit Service Factor	.0	.0	38.5
20. Unit Availability Factor	.0	.0	38.5
21. Unit Capacity Factor (MDC Net)	.0	.0	30.1
22. Unit Capacity Factor (DER Net)	.0	.0	28.6
23. Unit Forced Outage Rate	.0	.0	46.4

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 07/22/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	12/01/79	S	744.0	D	4		ZZ	ZZZZZZ	UNIT SHUTDOWN FOR MAJOR MODIFICATIONS AS REQUIRED BY THE NUCLEAR REGULATORY COMMISSION, INCLUDING NRC BULLETINS IEB 79-02 AND 79-14.

SUMMARY

THE PLANT REMAINS SHUTDOWN FOR MAJOR MODIFICATIONS AS ORDERED BY THE NRC.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3- Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

BIG ROCK POINT 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Michigan/Charlevoix 4 miles NE of Charlevoix
2.	Electric Reliability Council	East Central Area Reliability Coordination Agreement
3.	Type of Reactor	BWR
4.	Power Level (MWT)	240
5.	Design Electrical Rating (MWE Net)	72
6.	Date of Initial Criticality	September 27, 1962
7.	Date of Commercial Operation	March 29, 1963
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10.	Licensee	Consumers Power Company
11.	Corporate Address	212 West Michigan Avenue Jackson, Michigan 49201
12.	Corporate Contact	R. B. DeWitt
13.	Architect/Engineer	Bechtel Corporation
14.	Nuclear Steam Supply System	GE
15.	Constructor	Bechtel Corporation

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	K. R. BAKER
19.	IE Resident Inspector	G. Wright
20.	Licensing Project Manager	W. A. Paulson
21.	Docket Number	50-155
22.	License Number/Date of Issue	DPR-6 issued 8/30/62
23.	Public Document Room	Charlevoix Public Library 107 Clinton Street Charlevoix, Michigan

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

Operating routinely.

LAST IE SITE INSPECTION DATE: May 5-8, 1980

INSPECTION REPORT NO: 80-06

SUMMARY

INSPECTION SUMMARY

Inspection on April 14-18, (80-05): Included a review of the Security Plan; Security Program Audit; Security Organization; Protected Area Physical Barriers; Vital Area Physical Barriers; Lighting; Access Control (Identification, Authorization and Badging); Access Control (Search); Access Control (Escorting); Access Control (Vital Areas); Alarm Stations; Detection Aids; Assessment Aids; Communications; Testing and Maintenance; Compensatory Measures; Power Supply; Response; Locks, Keys and Combinations; Records and Reports; and Independent Inspection Effort. The inspection involved 73 inspector hours onsite by 2 NRC inspectors. The inspection began during the offshift; 18 inspection hours were accomplished during offshift periods. The licensee was found to be in compliance with NRC requirements within the 20 areas examined during the inspection except for the following 3 items: Assessment Aids - The licensee failed to provide adequate assessment aids that conform with approved security plan commitments; Access Control (Identification, Authorization and Badging) - The licensee did not conduct proper personnel screening in accordance with security plan commitments; and Locks, Keys and Combinations - The licensee failed to provide adequate key control.

Inspection on May 5-8, (80-06): Routine, unannounced inspection pertaining to followup of items of NUREG 0578 identified during meeting on February 28-29, 1980 between DOR and the licensee (refer to Inspection Report 80-03). The inspection involved 17 inspector hours onsite by one NRC inspector. Of the areas inspected, no items of noncompliance were identified.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

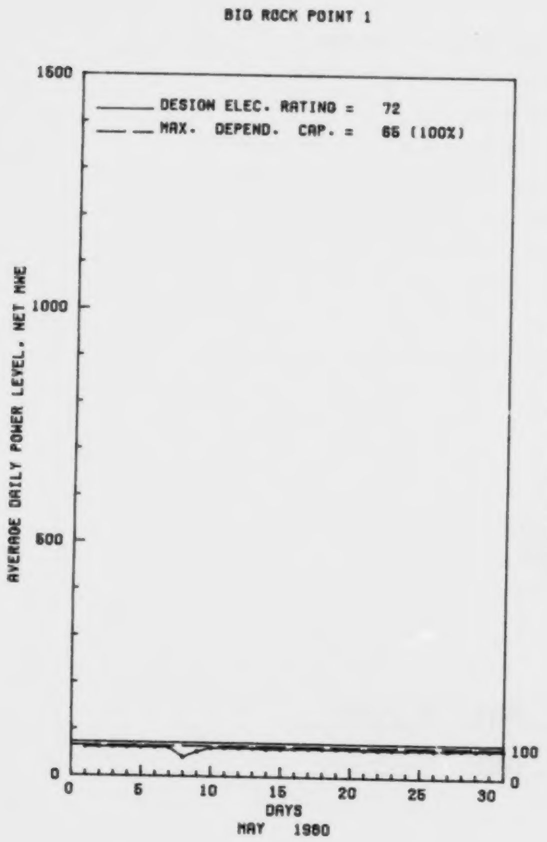
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-06/03L-0	04/03/80	05/01/80	Sensor channel D of RDS removed from service.
80-08/03L-0	04/11/80	05/01/80	Following auto test circuit alarms, the sensor channel D of the RDS was removed from service to correct the problem.
80-10/03L-0	04/23/80	05/23/80	Following intermittent automatic test circuit alarms, the sensor channel D of the RDS was removed from service to correct the problem.
80-12/03L-0	05/07/80	05/23/80	Following intermittent automatic test circuit alarms, the sensor channel D of the RDS was removed from service to correct the problem.
80-13/01T-0	05/09/80	05/23/80	A design review of two 24 inch ventilation isolation valves for the reactor containment indicates that damage to the air operator mechanism might occur during the pressure transient associated with the MCA described in the FSAR.

REVIEWED BY OMPA LOEB: L. Tomson DATE: 7/3/80

REVIEWED BY NRR DOR: J J Shea DATE: 7/6/80

BIG ROCK POINT 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-155
2. Utility Contact: G. Mimebauch (616) 547-6537 x180
3. Licensed Thermal Power (MWt): 240
4. Nameplate Rating (Gross MWe): 70.6 x 0.85 = 60
5. Design Electrical Rating (Net MWe): 72
6. Maximum Dependable Capacity (Gross MWe): 67
7. Maximum Dependable Capacity (Net MWe): 65
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): 65
10. Reasons for Restrictions, If Any: FUEL THERMAL HYDRAULIC LIMITS.

ITEM 21 CALCULATED WITH A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	3,647.0	150,570.0
12. Hours Reactor Was Critical	744.0	3,325.2	103,458.2
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	744.0	3,312.5	101,311.4
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	154,321	695,366	18,813,297
17. Gross Electrical Energy (MWH)	47,194	211,987	5,947,905
18. Net Electrical Energy (MWH)	44,631	200,314	5,628,651
19. Unit Service Factor	100.0	90.8	67.3
20. Unit Availability Factor	100.0	90.8	67.3
21. Unit Capacity Factor (MDC Net)	92.3	85.4	56.2
22. Unit Capacity Factor (DER Net)	83.3	76.3	51.9
23. Unit Forced Outage Rate	.0	1.2	24.1

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): OCTOBER 1980 - REFUELING - 6 TO 8 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
6	05/09/80	F	.0	A	5				POWER REDUCTION DUE TO LOSS OF REACTOR RECIRCULATION PUMP.

SUMMARY

THE UNIT OPERATED AT FULL POWER EXCEPT FOR A REDUCTION ON 05/08/80 AND 05/09/80 DUE TO LOSS OF RECIRC PUMP.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

BROWNS FERRY 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1. Location (state/county) (nearest population center)	Alabama/Limestone 10 miles NW of Decatur
2. Electric Reliability Council	Southeastern Electric Reliability Council
3. Type of Reactor	BWR
4. Power Level (MWT)	3293
5. Design Electrical Rating (MWE Net)	1065
6. Date of Initial Criticality	August 17, 1973
7. Date of Commercial Operation	August 1, 1974
8. Condenser Cooling Method	Once Through With Forced Draft Cooling Towers
9. Condenser Cooling Water Source	Tennessee River

Utility & Contractor Information

10. Licensee	Tennessee Valley Authority
11. Corporate Address	500A Chestnut Street Tower II Chattanooga, Tennessee 37401
12. Corporate Contact	H. G. Parris, Manager of Power.
13. Architect/Engineer	TVA
14. Nuclear Steam Supply System	GE
15. Constructor	TVA

Regulatory Information

16. IE Region responsible	II
17. IE Branch Chief	R. C. Lewis
18. IE Section Chief	H. C. Dance
19. IE Resident Inspectur	R. Sullivan (205) 729-6196
20. Licensing Project Manager	R. Clark
21. Docket Number	50-259
22. License Number/Date of Issue	DPR-33, 6/26/73
23. Public Document Room	Athens Public Library South and Forrest Athens, Alabama 35611

INSPECTION STATUS

ENFORCEMENT STATUS

AS REQUIRED BY 10 CFR 50, APPENDIX E, ITEM IV.H AND SECTION XI.A OF THE BROWNS FERRY SITE RADIOLOGICAL EMERGENCY PLAN, PERTINENT TRAINING SHALL BE PROVIDED TO INDIVIDUALS HAVING SPECIFIC DUTIES AND RESPONSIBILITIES IN AN EMERGENCY. TWO OF THE FOUR INDIVIDUALS DESIGNATED TO SUCCEED TO THE POSITION OF EMERGENCY DIRECTOR HAVE NOT RECEIVED TRAINING IN THE DUTIES AND RESPONSIBILITIES OF THAT POSITION.
(8016 INFRACTION)

AS REQUIRED BY TECHNICAL SPECIFICATION 6.3.A.8, DETAILED WRITTEN PROCEDURES COVERING, IN PART, IMPLEMENTATION OF THE RADIOLOGICAL EMERGENCY PLAN (REP)SHALL BE PREPARED, APPROVED, AND ADHERED TO. 1. SIX TRAINING ATTENDANCE RECORDS, FORM BF-45 FOR TRAINING HELD DURING JANUARY 8-15, 1980, DID NOT CONTAIN THE LESSON PLAN REFERENCE OR COURSE OUTLINE SUBJECT AS SPECIFIED IN BF 4.4. 2. THE CRITIQUE FOR THE JULY 1979 DRILL WAS NOT SUBMITTED TO THE PLANT SUPERINTENDENT WITHIN THE TIME LIMIT ALLOWED BY BF 22.1. THE EDNEY BUILDING RADIO TEST REQUIRED BY SECTION IV OF NUC PR-REP, WAS NOT COMPLETED AND/OR RECORDED ON 36 OCCASIONS DURING THE PERIOD JANUARY 1 - JUNE 30, 1979. 4. NO PROCEDURES HAVE BEEN PREPARED FOR : (A) INVENTORY OF EMERGENCY EQUIPMENT AND SUPPLIES, TRAINING OF OUTSIDE SUPPORT GROUPS AND UPDATING AND DISTRIBUTION OF THE REP AT THE MUSCLE SHOALS FACILITY. (B) TRAINING AND RETRAINING OF THE EMERGENCY TEAM LEADERS AND INVENTORY OF EMERGENCY REFERENCES AND SUPPLIES AT THE EDNEY BUILDING. (C) TRAINING AND RETRAINING OF DESIGNATED EMERGENCY DIRECTORS AT THE BROWNS FERRY SITE.
(8016 INFRACTION)

CRITERION IX OF APPENDIX B TO 10 CFR 50 STATES THAT MEASURES SHALL BE ESTABLISHED TO ASSURE THAT SPECIAL PROCESSES, INCLUDING WELDING, HEAT TREATING, AND NONDESTRUCTIVE TESTING, ARE CONTROLLED AND ACCOMPLISHED BY QUALIFIED PERSONNEL USING QUALIFIED PROCEDURES IN ACCORDANCE WITH APPLICABLE CODES, STANDARDS, SPECIFICATIONS, CRITERIA, AND OTHER SPECIAL REQUIREMENTS. THE ACCEPTED QUALITY ASSURANCE PROGRAM (TOPICAL REPORT TVA-TR-75-1A), SECTION 17.1A.9 STATES IN PART THAT TVA QA/QC PROCEDURES REQUIRE THAT SPECIAL PROCESSES ARE TO BE CONTROLLED AND ACCOMPLISHED BY QUALIFIED PERSONNEL USING QUALIFIED WRITTEN PROCEDURES. IN ADDITION, SECTION 6.3.A.1 OF THE FACILITY TECHNICAL SPECIFICATION STATED THAT: DETAILED WRITTEN PROCEDURES, INCLUDING APPLICABLE CHECKOFF LISTS COVERING ITEMS LISTED BELOW SHALL BE PREPARED, APPROVED AND ADHERED TO FOR PREVENTIVE OR CORRECTIVE MAINTENANCE OPERATIONS WHICH COULD HAVE AN EFFECT ON THE SAFETY OF THE REACTOR. ON MARCH 11, 1980, QUALIFIED PROCEDURES WERE NOT USED ON THE HPCI STEAM LINE WHEN ATTEMPTING TO CORRECT A MISALIGNMENT PROBLEM ON THE HPCI STEAM LINE BY USE OF HEAT TREATING.
(8019 INFRACTION)

10 CFR 50.72(A)(7) REQUIRES THAT EACH LICENSEE NOTIFY THE NRC OPERATIONS CENTER AS SOON AS POSSIBLE AS IN ALL CASES WITHIN ONE HOUR BY TELEPHONE OF THE OCCURRENCE OF A SIGNIFICANT EVENT. CONTRARY TO THE ABOVE A PROMPT REPORT WAS NOT MADE FOLLOWING THE AUTOMATIC ACTUATION OF THE "B" DIESEL GENERATOR AT 1:00 P.M. ON APRIL 23, 1980.
(8020 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

100 PERCENT FULL POWER OPERATION.

LAST IE SITE INSPECTION DATE: APRIL 5 - MAY 2, 1980

INSPECTION REPORT NO: 50-259/80-20

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-001/ 01T-1	01/07/80	01/18/80	DIESEL GENERATOR LOSS OF SPEED CONTROL.
80-011/ 01T-1	02/01/80	02/14/80	PURGE SYSTEM DESIGN DEFICIENCY.
80-021/ 03L-0	03/01/80	03/28/80	WIND SPEED INSTRUMENTATION FAILED TO INDICATE TRUE WIND SPEED.
80-023/ 03L-0	03/12/80	04/08/80	HPCI REACTOR HIGH WATER LEVEL TRIP INOPERABLE DUE TO WIRING ERROR.
80-024/ 03L-0	03/12/80	04/10/80	161KV BREAKERS TRIP CAUSE BY LOOSE BRACKET.
80-025/ 03L-0	03/13/80	04/11/80	DURING PRIMARY CONTAINMENT ISOLATION TESTING VALVE 74-76 FAILED TO CLOSE.
80-026/ 03L-0	03/22/80	04/18/80	HIGH PRESSURE COOLANT INJECTION STOP VALVE FCV-73-18 WOULD NOT OPEN FULLY.
80-027/ 03L-0	03/22/80	04/18/80	RESIDUAL HEAT REMOVAL SERVICE WATER PUMP TIMER OUT OF SPECIFICATION.
80-028/ 03L-0	04/03/80	05/01/80	SCRAM TIME TEST CONDUCTED AT 950 PSIA VICE 950 PSIG.
80-29/ 03L-0	04/07/80	05/06/80	HPCI TURBINE TRIPPED DUE TO FAILED RUPTURE DISC.
80-30/ 03L-0	04/06/80	05/05/80	DAMAGED HIGH TEMP SWITCH TS-73-2A.
80-037/ 03L-0	04/02/80	05/01/80	REACTOR CORE ISOLATION COOLING TRIPPED DUE TO HIGH RUPTURE DISC PRESSURE.

SUMMARY

INSPECTION SUMMARY

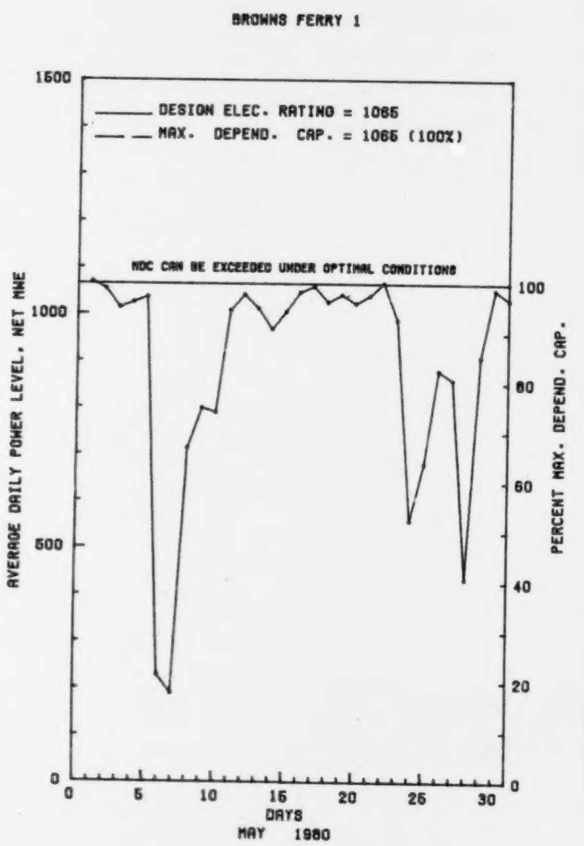
INSPECTION APRIL 5 - MAY 2 (80-20): THIS ROUTINE INSPECTION INVOLVED 33 RESIDENT INSPECTOR-HOURS IN THE AREAS OF OPERATIONAL SAFETY, REPORTABLE OCCURRENCES, IE BULLETIN FOLLOWUP, PLANT PHYSICAL PROTECTION, SURVEILLANCE INSTRUCTIONS, PLANT OPERATIONS REVIEW COMMITTEE, AND CONTAINMENT PURGING DURING OPERATION. OF THE SEVEN AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED IN FIVE AREAS. ONE ITEM OF APPARENT NONCOMPLIANCE WAS FOUND IN EACH OF TWO AREAS (INFRACTION - FAILURE TO FOLLOW PROCEDURE WHILE PERFORMING SURVEILLANCE); AND (INFRACTION - FAILURE TO MAKE A PROMPT REPORT TO NRC).

REVIEWED BY OMPA LOEB: Earl [illegible] **DATE:** 7/3/80

REVIEWED BY NRR DOR: Dick Clark **DATE:** 07-07-80

BROWNS FERRY 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-259
2. Utility Contact: Don Green (205) 729-6846
3. Licensed Thermal Power (MWt): 3293
4. Nameplate Rating (Gross MWe): 1280 x 0.9 = 1152
5. Design Electrical Rating (Net MWe): 1065
6. Maximum Dependable Capacity (Gross MWe): 1098
7. Maximum Dependable Capacity (Net MWe): 1065
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	3,647.0	51,145.0
12. Hours Reactor Was Critical	708.0	1,531.6	29,808.9
13. Reactor Reserve Shutdown Hours	35.9	283.8	4,973.3
14. Hours Generator On-Line	701.2	1,428.5	29,041.4
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	2,075,263	3,816,795	77,953,288
17. Gross Electrical Energy (MWH)	680,430	1,253,390	25,771,850
18. Net Electrical Energy (MWH)	663,602	1,213,207	25,015,175
19. Unit Service Factor	94.2	39.2	56.8
20. Unit Availability Factor	94.2	39.2	56.8
21. Unit Capacity Factor (MDC Net)	83.8	31.2	45.9
22. Unit Capacity Factor (DER Net)	83.8	31.2	45.9
23. Unit Forced Outage Rate	5.8	18.8	32.4

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
122	05/06/80	F	24.4	B	3				GENERATOR LOAD REJECTION (NO. 6 INTERMEDIATE VALVE CLOSED WHILE TESTING NO. 5 INTERMEDIATE VALVE.
123	05/07/80	F	7.7	A	3				TURBINE TRIP DUE TO A BLOWN FUSE IN "A" FEEDWATER INVERTER AND "B" FEEDWATER HIGH LEVEL SIGNAL.
124	05/23/80	S	.0	H	5				DERATED FOR CONTROL ROD SEQUENCE EXCHANGE.
125	05/27/80	F	10.7	G	3				GENERATOR LOAD REJECTION WHILE REPLACING A PK BLOCK IN MAIN TRANSFORMER CREATING A DIFFERENTIAL AND LOAD REJECTION (PERSONNEL ERROR).

SUMMARY

THE UNIT OPERATED ROUTINELY DURING MAY WITH SCRAMS ON 05/06/80, DUE TO EHC PRESSURE FLUCTUATIONS, ON 05/07/80 DUE TO FALSE HIGH WATER LEVEL SIGNALS, AND ON 05/27/80 AFTER LOAD REJECTION CAUSED BY PERSONNEL ERROR.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H; Same source.

BROWNS FERRY 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Alabama/Limestone 10 miles NW of Decatur
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	3293
5.	Design Electrical Rating (MWE Net)	1065
6.	Date of Initial Criticality	July 20, 1974
7.	Date of Commercial Operation	March 1, 1975
8.	Condenser Cooling Method	Once Through With Forced Draft Cooling Towers
9.	Condenser Cooling Water Source	Tennessee River

Utility & Contractor Information

10.	Licensee	Tennessee Valley Authority
11.	Corporate Address	500A Chestnut Street Tower II Chattanooga, Tennessee 37401
12.	Corporate Contact	H. G. Parris, Manager of Power
13.	Architect/Engineer	TVA
14.	Nuclear Steam Supply System	GE
15.	Constructor	TVA

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. Lewis
18.	IE Section Chief	H. C. Dance
19.	IE Resident Inspector	R. Sullivan (205) 729-6196
20.	Licensing Project Manager	R. Clark
21.	Docket Number	50-260
22.	License Number/Date of Issue	DPR-52, 6/28/74
23.	Public Document Room	Athens Public Library South and Forrest Athens, Alabama 35611

INSPECTION STATUS

ENFORCEMENT STATUS

AS REQUIRED BY 10 CFR 50, APPENDIX E, ITEM IV.H AND SECTION XI.A OF THE BROWNS FERRY SITE RADIOLOGICAL EMERGENCY PLAN, PERTINENT TRAINING SHALL BE PROVIDED TO INDIVIDUALS HAVING SPECIFIC DUTIES AND RESPONSIBILITIES IN AN EMERGENCY. TWO OF THE FOUR INDIVIDUALS DESIGNATED TO SUCCEED TO THE POSITION OF EMERGENCY DIRECTOR HAVE NOT RECEIVED TRAINING IN THE DUTIES AND RESPONSIBILITIES OF THAT POSITION.
(8013 INFRACTION)

AS REQUIRED BY TECHNICAL SPECIFICATION 6.3.A.8, DETAILED WRITTEN PROCEDURES COVERING, IN PART, IMPLEMENTATION OF THE RADIOLOGICAL EMERGENCY PLAN (REP)SHALL BE PREPARED, APPROVED, AND ADHERED TO. 1. SIX TRAINING ATTENDANCE RECORDS, FORM BF-45 FOR TRAINING HELD DURING JANUARY 8-15, 1980, DID NOT CONTAIN THE LESSON PLAN REFERENCE OR COURSE OUTLINE SSUBJECT AS SPECIFIED IN BF 4.4. 2. THE CRITIQUE FOR THE JULY 1979 DRILL WAS NOT SUBMITTED TO THE PLANT SUPERINTENDENT WITHIN THE TIME LIMIT ALLOWED BY BF 22.1. THE EDNEY BUILDING RADIO TEST REQUIRED BY SECTION IV OF NUC PR-REP, WAS NOT COMPLETED AND/OR RECORDED ON 36 OCCASIONS DURING THE PERIOD JANUARY 1 - JUNE 30, 1979. 4. NO PROCEDURES HAVE BEEN PREPARED FOR: (A) INVENTORY OF EMERGENCY EQUIPMENT AND SUPPLIES, TRAINING OF OUTSIDE SUPPORT GROUPS AND UPDATING AND DISTRIBUTION OF THE REP AT THE MUSCLE SHOALS FACILITY. (B) TRAINING AND RETRAINING OF THE EMERGENCY TEAM LEADERS AND INVENTORY OF EMERGENCY REFERENCES AND SUPPLIES AT THE EDNEYBUILDING. (C) TRAINING AND RETRAINING OF DESIGNATED EMERGENCY DIRECTORS AT THE BROWNS FERRY SITE.
(8013 INFRACTION)

10 CFR 50.72(A)(7) REQUIRES THAT EACH LICENSEE NOTIFY THE NRC OPERATIONS CENTER AS SOON AS POSSIBLE AS IN ALL CASES WITHIN ONE HOUR BY TELEPHONE OF THE OCCURRENCE OF A SIGNIFICANT EVENT. CONTRARY TO THE ABOVE A PROMPT REPORT WAS NOT MADE FOLLOWING THE AUTOMATIC ACTUATION OF THE "B" DIESEL GENERATOR AT 1:00 P.M. ON APRIL 23, 1980.
(8015 INFRACTION)

ITEM 6.3.A.1 OF THE FACILITY TECHNICAL SPECIFICATION STATES THAT DETAILED WRITTEN PROCEDURES, INCLUDING CHECKOFF LISTS, SHALL BE PREPARED, APPROVED AND ADHERED TO FOR SURVEILLANCE AND TESTING REQUIREMENTS. CONTRARY TO THE ABOVE, ON APRIL 22, 1980, SURVEILLANCE INSTRUCTION 4.1.A-7, REACTOR PROTECTION SYSTEM REACTOR WATER LEVEL, WAS NOT ADHERED TO DURING IT'S PERFORMANCE OF VALVING MANIPULATIONS.
(8015 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

100 PERCENT FULL POWER OPERATION.

LAST IE SITE INSPECTION DATE: APRIL 5 - MAY 2, 1980

INSPECTION REPORT NO: 50-260/80-15

SUMMARY

INSPECTION SUMMARY

INSPECTION APRIL 5 - MAY 2 (80-15): THIS ROUTINE INSPECTION INVOLVED 33 RESIDENT INSPECTOR-HOURS IN THE AREAS OF OPERATIONAL SAFETY, REPORTABLE OCCURRENCES, IE BULLETIN FOLLOWUP, PLANT PHYSICAL PROTECTION, SURVEILLANCE INSTRUCTIONS, PLANT OPERATIONS REVIEW COMMITTEE, AND CONTAINMENT PURGING DURING OPERATION. OF THE SEVEN AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED IN FIVE AREAS. ONE ITEM OF APPARENT NONCOMPLIANCE WAS FOUND IN EACH OF TWO AREAS (INFRACTION - FAILURE TO FOLLOW PROCEDURE WHILE PERFORMING SURVEILLANCE); AND (INFRACTION - FAILURE TO MAKE A PROMPT REPORT TO NRC).

REPORTS RECEIVED FROM LICENSEE

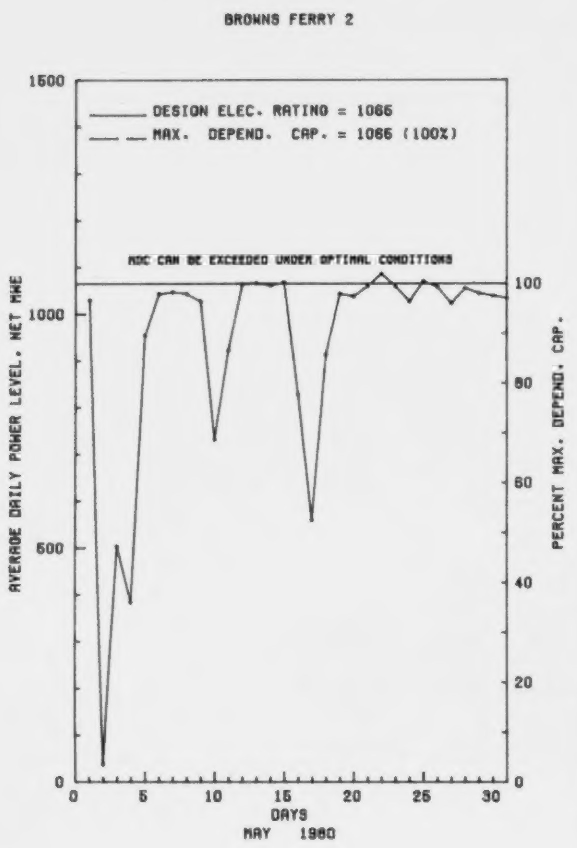
NUMBER	DATE OR EVENT	DATE OF REPORT	SUBJECT
80-013/ 03L-0	03/01/80	03/28/80	161KV POWER SUPPLY LOST INTERMITTENTLY DURING STORM.
80-014/ 03L-0	02/16/80	03/31/80	SURVEILLANCE PROCEDURE 42-B-44 FOUND INADEQUATE DURING TESTING.
80-015/ 03L-0	03/09/80	04/07/80	ROD BLOCK MONITOR A BYPASSED MORE THAN 24 HOURS IN 30 DAY PERIOD.
80-016/ 03L-0	03/10/80	04/08/80	HALF ISOLATION OCCURRED DUE TO BLOCK SENSING LINE TO PS-1-86.
80-017/ 03L-0	03/10/80	04/08/80	FAILED RUPTURED DISC IN TURBINE EXHAUST LINE RENDERED HPCI INOPERABLE.
80-018/ 03L-0	03/21/80	04/18/80	2A REACTOR RECIRCULATION MG SET TRIPPED STATOR HI TEMPERATURE.
80-019/ 03L-0	04/13/80	05/01/80	AIR PARTICULATE MONITOR DRIFTED HIGH.

REVIEWED BY OMPA LOEB: Earl Hill **DATE:** 7-3-80

REVIEWED BY NRR DOR: Dick Clark **DATE:** 07-07-80

BROWNS FERRY 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-260
2. Utility Contact: Don Green (205) 729-6846
3. Licensed Thermal Power (MWt): 3293
4. Nameplate Rating (Gross MWe): 1280 x 0.9 = 1152
5. Design Electrical Rating (Net MWe): 1065
6. Maximum Dependable Capacity (Gross MWe): 1098
7. Maximum Dependable Capacity (Net MWe): 1065
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	3,647.0	46,056.0
12. Hours Reactor Was Critical	715.5	3,146.6	27,631.0
13. Reactor Reserve Shutdown Hours	28.5	500.4	12,275.3
14. Hours Generator On-Line	700.2	3,067.7	26,732.2
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	2,120,551	8,781,400	74,936,400
17. Gross Electrical Energy (MWH)	711,810	2,936,690	24,886,758
18. Net Electrical Energy (MWH)	694,101	2,858,057	24,189,526
19. Unit Service Factor	94.1	84.1	58.0
20. Unit Availability Factor	94.1	84.1	58.0
21. Unit Capacity Factor (MDC Net)	87.6	73.6	49.3
22. Unit Capacity Factor (DER Net)	87.6	73.6	49.3
23. Unit Forced Outage Rate	5.9	15.9	34.9

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
REFUEL OUTAGE SEPTEMBER, 1980
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
136	05/02/80	F	20.2	A	2				TO ACCOMODATE REPAIRS TO STEAM LEAK IN STEAM TUNNEL.
137	05/03/80	F	14.8	A	2				TO ACCOMODATE REPAIRS TO NO. 1 CONTROL VALVE.
138	05/09/80	S	.0	B	5				DERATED FOR CONTROL ROD PATTERN ADJUSTMENT.
139	05/16/80	F	8.8	A	2				TO ACCOMMODATE REPAIRS TO EHC LEAK (NO. 3 SERVO VALVE).

SUMMARY

THE REACTOR OPERATED NEAR FULL POWER EXCEPT FOR TRIPS OCCURRING ON 05/02/80 AND ON 05/03/80 FOR CONTROL VALVE REPAIR, AND ON 05/16/80 FOR REPAIR OF A CONTROL VALVE SERVO-MOTOR.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

BROWNS FERRY 3

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Alabama/Limestone 10 miles NW of Decatur
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	3293
5.	Design Electrical Rating (MWE Net)	1065
6.	Date of Initial Criticality	August 8, 1976
7.	Date of Commercial Operation	March 1, 1977
8.	Condenser Cooling Method	Once Through With Forced Draft Cooling Tower
9.	Condenser Cooling Water Source	Tennessee River

Utility & Contractor Information

10.	Licensee	Tennessee Valley Authority
11.	Corporate Address	500A Chestnut Street Tower II Chattanooga, Tennessee 37401
12.	Corporate Contact	H. G. Parris, Manager of Power
13.	Architect/Engineer	TVA
14.	Nuclear Steam Supply System	GE
15.	Constructor	TVA

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R.C. Lewis
18.	IE Section Chief	H. C. Dance
19.	IE Resident Inspector	R. Sullivan (205) 729-6196
20.	Licensing Project Manager	R. Clark
21.	Docket Number	50-296
22.	License Number/Date of Issue	DPR-68, 7/2/76
23.	Public Document Room	Athens Public Library South and Forrest Athens, Alabama 35611

INSPECTION STATUS

ENFORCEMENT STATUS

AS REQUIRED BY TECHNICAL SPECIFICATIONS 3.5.F.2, THE RCICS MAY BE INOPERABLE FOR A PERIOD NOT TO EXCEED 7 DAYS OF THE HPCIS IS OPERABLE DURING SUCH TIME PERIOD. IN ADDITION, BROWNS FERRY STANDARD PRACTICE 12.2 REQUIRES THE SHIFT ENGINEER, ASSISTANT SHIFT ENGINEER AND UNIT OPERATOR TO KEEP JOURNALS AND THESE JOURNALS SHALL CONTAIN SIGNIFICANT MALFUNCTIONS. ON DECEMBER 8, 1979, RCICS WAS INOPERABLE FOR APPROXIMATELY 4 HOURS FOR REPLACEMENTS AND CALIBRATION OF THE EGM BOX AND HPCIS WAS NOT DETERMINED TO BE OPERABLE. IN ADDITION, NEITHER THE UNIT OPERATOR, ASSISTANT SHIFT ENGINEER NOR THE SHIFT ENGINEER DOCUMENTED IN THEIR JOURNALS THAT RCICS WAS INOPERABLE FOR REPLACEMENT AND CALIBRATION OF OF THE EGM BOX.
(8012 INFRACTION)

AS REQUIRED BY 10 CFR 50, APPENDIX E, ITEM IV.H AND SECTION XI.A OF THE BROWNS FERRY SITE RADIOLOGICAL EMERGENCY PLAN, PERTINENT TRAINING SHALL BE PROVIDED TO INDIVIDUALS HAVING SPECIFIC DUTIES AND RESPONSIBILITIES IN AN EMERGENCY. TWO OF THE FOUR INDIVIDUALS DESIGNATED TO SUCCEED TO THE POSITION OF EMERGENCY DIRECTOR HAVE NOT RECEIVED TRAINING IN THE DUTIES AND RESPONSIBILITIES OF THAT POSITION.
(8014 INFRACTION)

AS REQUIRED BY TECHNICAL SPECIFICATION 6.3.A.8, DETAILED WRITTEN PROCEDURES COVERING, IN PART, IMPLEMENTATION OF THE RADIOLOGICAL EMERGENCY PLAN (REP)SHALL BE PREPARED, APPROVED, AND ADHERED TO. 1. SIX TRAINING ATTENDANCE RECORDS, FORM BF-45 FOR TRAINING HELD DURING JANUARY 8-15, 1980, DID NOT CONTAIN THE LESSON PLAN REFERENCE OR COURSE OUTLINE SUBJECT AS SPECIFIED IN BF 4.4. 2. THE CRITIQUE FOR THE JULY 1979 DRILL WAS NOT SUBMITTED TO THE PLANT SUPERINTENDENT WITHIN THE TIME LIMIT ALLOWED BY BF 22.1. THE EDNEY BUILDING RADIO TEST REQUIRED BY SECTION IV OF NUC PR-REP, WAS NOT COMPLETED AND/OR RECORDED ON 36 OCCASIONS DURING THE PERIOD JANUARY 1 - JUNE 30, 1979. 4. NO PROCEDURES HAVE BEEN PREPARED FOR: (A) INVENTORY OF EMERGENCY EQUIPMENT AND SUPPLIES, TRAINING OF OUTSIDE SUPPORT GROUPS AND UPDATING AND DISTRIBUTION OF THE REP AT THE MUSCLE SHOALS FACILITY. (B) TRAINING AND RETRAINING OF THE EMERGENCY TEAM LEADERS AND INVENTORY OF EMERGENCY REFERENCES AND SUPPLIES AT THE EDNEY BUILDING. (C) TRAINING AND RETRAINING OF DESIGNATED EMERGENCY DIRECTORS AT THE BROWNS FERRY SITE.
(8014 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

FULL POWER OPERATIONS.

LAST IE SITE INSPECTION DATE: APRIL 5 - MAY 2, 1980

INSPECTION REPORT NO: 50-296/80-16

SUMMARY

INSPECTION SUMMARY

INSPECTION APRIL 5 - MAY 2 (80-16): THIS ROUTINE INSPECTION INVOLVED 34 RESIDENT INSPECTOR-HOURS IN THE AREAS OF OPERATIONAL SAFETY, REPORTABLE OCCURRENCES, IE BULLETIN FOLLOWUP, PLANT PHYSICAL PROTECTION, SURVEILLANCE INSTRUCTIONS, PLANT OPERATIONS REVIEW COMMITTEE, AND CONTAINMENT PURGING DURING OPERATION. OF THE SEVEN AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED IN FIVE AREAS. ONE ITEM OF APPARENT NONCOMPLIANCE WAS FOUND IN EACH OF TWO AREAS (INFRACTION - FAILURE TO FOLLOW PROCEDURE WHILE PERFORMING SURVEILLANCE); AND (INFRACTION FAILURE TO MAKE A PROMPT REPORT TO NRC).

REPORTS RECEIVED FROM LICENSEE

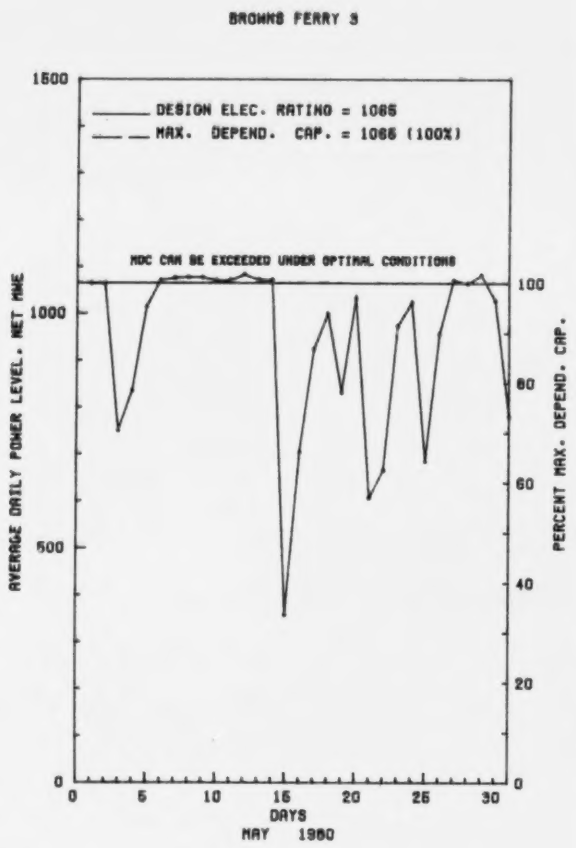
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
78-008/ 01-1	04/15/78	04/08/80	RELIEF VALVE FAILED TO RESEAT.
80-007/ 03L-0	03/01/80	03/28/80	161KV POWER SUPPLY LOST MOMENTARILY DURING STORM.
80-008/ 03L-0	04/02/80	05/01/80	3B DIESEL GENERATOR GOVERNOR MALFUNCTION.
80-009/ 03L-0	04/04/80	05/02/80	RELAY OVERHEATS CAUSES BLOWN FUSE RESULTS IN CONTAINMENT ISOLATION VALVES TO SHUT.

REVIEWED BY OMPA LOEB: Earl Hill **DATE:** 7/3/80

REVIEWED BY NRR DOR: Dick Clark **DATE:** 07-07-80

BROWNS FERRY 3

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-296
2. Utility Contact: Don Green (205) 729-6846
3. Licensed Thermal Power (MWt): 3293
4. Nameplate Rating (Gross MWe): 1280 x 0.9 = 1152
5. Design Electrical Rating (Net MWe): 1065
6. Maximum Dependable Capacity (Gross MWe): 1098
7. Maximum Dependable Capacity (Net MWe): 1065
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	3,647.0	28,511.0
12. Hours Reactor Was Critical	723.5	3,214.9	22,122.6
13. Reactor Reserve Shutdown Hours	20.5	20.5	1,678.9
14. Hours Generator On-Line	718.5	3,186.4	21,624.2
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	2,139,984	9,947,427	63,855,293
17. Gross Electrical Energy (MWH)	718,180	3,356,390	20,758,970
18. Net Electrical Energy (MWH)	700,399	3,272,840	20,160,613
19. Unit Service Factor	95.6	87.4	75.8
20. Unit Availability Factor	96.6	87.4	75.8
21. Unit Capacity Factor (MDC Net)	88.4	84.3	66.4
22. Unit Capacity Factor (DER Net)	88.4	84.3	66.4
23. Unit Forced Outage Rate	3.4	12.6	9.6

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
REFUEL OUTAGE OCTOBER, 1980.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
123	05/03/80	F	.0	B	5				DERATED FOR MAINTENANCE ON "B" RECIRCULATION PUMP.
124	05/15/80	F	14.5	G	3				APPARENT GENERATOR FIELD GROUND (PERSONNEL ERROR).
125	05/18/80	S	.0	H	5				DERATED FOR CONTROL ROD PATTERN ADJUSTMENT.
126	05/21/80	F	11.0	B	3				REACTOR SCRAMMED WHILE PERFORMING REACTOR LOW WATER LEVEL SI.
127	05/24/80	S	.0	H	5				DERATED FOR CONTROL ROD PATTERN ADJUSTMENT.
128	05/30/80	S	.0	H	5				DERATED FOR CONTROL ROD SEQUENCE EXCHANGE.

SUMMARY

THE FACILITY RAN NORMALLY, EXCEPT FOR SCRAMS ON 05/13/80 DUE TO LOAD REJECTION, AND ON 05/21/80 DUE TO HIGH FLUX INDICATIONS.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

BRUNSWICK 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	North Carolina/Brunswick 3 miles N of Southport
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2436
5.	Design Electrical Rating (MWE Net)	821
6.	Date of Initial Criticality	October 8, 1976
7.	Date of Commercial Operation	March 18, 1977
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Cape Fear River

Utility & Contractor Information

10.	Licensee	Carolina Power and Light Company
11.	Corporate Address	411 Fayetteville Street Raleigh, North Carolina 27602
12.	Corporate Contact	J. A. Jones Executive Vice President
13.	Architect/Engineer	United Engineers & Constructors, Inc.
14.	Nuclear Steam Supply System	GE
15.	Constructor	Brown and Root, Inc.

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R.C. Lewis
18.	IE Section Chief	P.J. Kellogg
19.	IE Resident Inspector	J. E. Outzs (919) 457-9531
20.	Licensing Project Manager	J. Hannon
21.	Docket Number	50-325
22.	License Number/Date of Issue	DPR-71, 9/8/76
23.	Public Document Room	Southport-Brunswick County Library 109 W. Moore Street Southport, North Carolina 28461

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

UNIT DOWN MAY 26, 1980 FOR REFUELING.

LAST IE SITE INSPECTION DATE: APRIL 28 - MAY 1 AND 6-8, 1980

INSPECTION REPORT NO: 50-325/80-20

SUMMARY

INSPECTION SUMMARY

INSPECTION MARCH 31 - MAY 2 (80-17): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 56 INSPECTOR-HOURS ON SITE IN THE AREAS OF CONTROL ROOM OBSERVATIONS, REVIEW OF OPERATOR LOGS, TAGOUT OF SYSTEMS, TOURS OF VITAL AREAS, PHYSICAL SECURITY, RADIATION PROTECTION, HOUSEKEEPING AND CLEANLINESS, FIRE PROTECTION, OBSERVATION OF MAINTENANCE, OBSERVATION OF SURVEILLANCE AND REVIEW AND FOLLOWUP ON LICENSEE EVENT REPORTS. THIS ROUTINE INSPECTION INVOLVED 56 MANHOURS OF INSPECTION TIME ON SITE IN TEN AREAS IDENTIFIED ABOVE. OF THE TEN AREAS INSPECTED NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION APRIL 28 - MAY 1 AND 6-8 (80-20): INCLUDED REVIEW OF SECURITY PLAN; PROTECTED AREA PHYSICAL BARRIERS; VITAL AREA PHYSICAL BARRIERS; LIGHTING; ACCESS CONTROL (IDENTIFICATION, AUTHORIZATION AND BADGING); ACCESS CONTROL (SEARCHING); ACCESS CONTROL (VITAL AREAS); DETECTION AIDS; COMMUNICATIONS; TESTING AND MAINTENANCE; COMPENSATORY MEASURES; POWER SUPPLY; RESPONSE; RECORDS AND REPORTS. THE INSPECTION INVOLVED 51 INSPECTOR-HOURS ON SITE BY TWO NRC INSPECTORS. EIGHT INSPECTOR-HOURS WERE ACCOMPLISHED DURING OFFSHIFT PERIODS. OF THE FOURTEEN AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE WERE DISCLOSED IN EIGHT AREAS. SIX APPARENT ITEMS OF NONCOMPLIANCE WERE IDENTIFIED IN THREE AREAS: (1) PHYSICAL BARRIERS - FAILURE TO MAINTAIN ISOLATION ZONE CLEAR OF OBJECTS; (2) PHYSICAL BARRIERS - INOPERATIVE LOCKING DEVICE ON PROTECTED AREA (PA) BARRIER; (3) ACCESS CONTROL - FAILURE TO SEARCH HAND-CARRIED PACKAGES; (4) ACCESS CONTROL - FAILURE TO FOLLOW PROCEDURES FOR VEHICLE ACCESS INTO PA; (5) DETECTION AIDS - INTRUSION ALARMS AT CERTAIN LOCATIONS ALONG THE PA INOPERATIVE; AND (6) DETECTION AIDS - TAMPER INDICATING DEVICES DID NOT PERFORM IN ACCORDANCE WITH SPECIFICATIONS.

REPORTS RECEIVED FROM LICENSEE

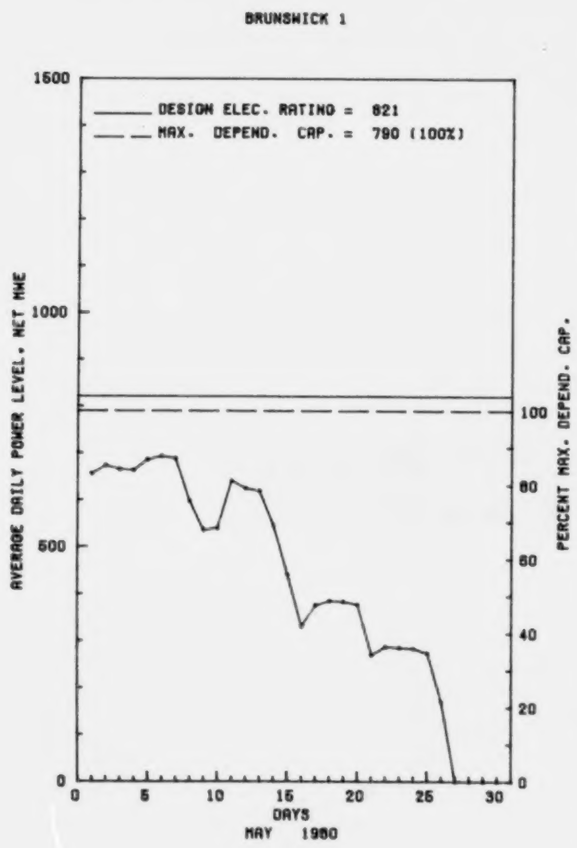
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-19/ 03L-0	03/18/80	04/15/80	INSTRUMENT DRIFT ON HI STEAM LINE RADIATION CHANNEL.
80-22/ 03L-0	03/18/80	04/15/80	ERRATIC CONTAINMENT ATMOSPHERE MONITOR CAUSED BY PARTIALLY CLOGGED INLET HOSE.
80-25/ 03L-0	04/05/80	04/24/80	CONTROL ROD 30-03 POSITION INDICATION FAILED.
80-27/ 03L-0	03/19/80	04/16/80	STAND BY GAS TREATMENT SYSTEM 1B INOPERABLE-POWER CIRCUIT BREAKER TRIPPED.
80-30/ 03L-0	03/24/80	04/21/80	CONTAINMENT ATMOSPHERE MONITOR IN LOW FLOW ALARM AND INOPERABLE.
80-31/ 03L-0	03/29/80	04/24/80	CONTAINMENT OXYGEN MONITOR FAILURES.
80-42/ 03L-0	04/16/80	05/07/80	HIGH STEAM LINE RADIATION CHANNEL OUT OF SPECIFICATION.

REVIEWED BY OMPA LOEB: Earl Dill **DATE:** 7/80

REVIEWED BY NRR DOR: John N. Hannon **DATE:** 7/7/80

BRUNSWICK 1

AVERAGE POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-325
2. Utility Contact: E. Willis (919) 457-9521
3. Licensed Thermal Power (MWt): 2436
4. Nameplate Rating (Gross MWe): 963 x 0.9 = 867
5. Design Electrical Rating (Net MWe): 821
6. Maximum Dependable Capacity (Gross MWe): 815
7. Maximum Dependable Capacity (Net MWe): 790
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	3,647.0	28,104.0
12. Hours Reactor Was Critical	615.3	3,270.4	20,650.0
13. Reactor Reserve Shutdown Hours	.0	.0	1,647.1
14. Hours Generator On-Line	615.3	3,148.3	19,481.1
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	965,202	6,237,934	60,067,430
17. Gross Electrical Energy (MWH)	316,244	2,087,474	13,326,293
18. Net Electrical Energy (MWH)	303,678	2,011,410	12,819,220
19. Unit Service Factor	82.7	86.3	69.3
20. Unit Availability Factor	82.7	86.3	69.3
21. Unit Capacity Factor (MDC Net)	51.7	69.8	57.7
22. Unit Capacity Factor (DER Net)	49.7	67.2	55.6
23. Unit Forced Outage Rate	.0	10.5	19.9

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 07/03/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
016	05/08/80	F	5.5	A	5		HC	HTEXCH	POWER WAS REDUCED TO PERFORM WORK ON THE CONDENSER WATER BOXES. 1A NORTH CONDENSER WAS REMOVED FROM SERVICE DUE TO PLUGGING. THIS WATER BOX WAS CLEANED AND RETURNED TO SATISFACTORY SERVICE.
017	05/09/80	F	13.9	A	5		HC	HTEXCH	POWER WAS REDUCED TO PERFORM WORK ON THE CONDENSER WATER BOXES. 1A SOUTH CONDENSER WAS REMOVED FROM SERVICE DUE TO PLUGGING. THE WATER BOX WAS CLEANED AND RETURNED TO SATISFACTORY SERVICE.
018	05/14/80	F	281.1	A	5		HC	HTEXCH	POWER WAS REDUCED BECAUSE OF CONDUCTIVITY ASSOCIATED WITH CONDENSER WATER BOX PROBLEMS. 1B NORTH CONDENSER WAS REMOVED FROM SERVICE DUE TO TUBE LEAKAGE.
	05/26/80	S	128.7	C	1		RC	FUELXX	REFUELING.

SUMMARY

THE UNIT OPERATED NEAR FULL POWER UNTIL END OF CYCLE COASTDOWN BEGAN. REDUCTIONS OCCURRED ON 05/08/80, ON 05/09/80 AND ON 05/14/80 DUE TO CONDENSER WATERBOX PROBLEMS. THE PLANT BEGAN REFUELING ON 5/26/80.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

BRUNSWICK 2

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	North Carolina/Brunswick 3 miles N of Southport
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2436
5.	Design Electrical Rating (MWE Net)	821
6.	Date of Initial Criticality	March 20, 1975
7.	Date of Commercial Operation	November 3, 1975
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Cape Fear River

Utility & Contractor Information

10.	Licensee	Carolina Power and Light Company
11.	Corporate Address	411 Fayetteville Street Raleigh, North Carolina 27602
12.	Corporate Contact	J. A. Jones Executive Vice President
13.	Architect/Engineer	United Engineers & Constructors, Inc.
14.	Nuclear Steam Supply System	GE
15.	Constructor	Brown and Root, Inc.

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R.C. Lewis
18.	IE Section Chief	P.J. Kellogg
19.	IE Resident Inspector	J. E. Outzs (919) 457-9531
20.	Licensing Project Manager	J. Hannon
21.	Docket Number	50-324
22.	License Number/Date of Issue	DPR-62, 12/27/74
23.	Public Document Room	Southport-Brunswick County Library 109 W. Moore Street Southport, North Carolina 28461

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

UNIT SHUTDOWN FOR REFUELING AND TORUS MODIFICATION.

LAST IE SITE INSPECTION DATE: APRIL 28 - MAY 1 AND 6-8, 1980

INSPECTION REPORT NO: 50-324/80-17

SUMMARY

INSPECTION SUMMARY

INSPECTION MARCH 31 - MAY 2 (80-14): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 50 INSPECTOR-HOURS ON SITE IN THE AREAS OF CONTROL ROOM OBSERVATIONS, REVIEW OF OPERATOR LOGS, TAGOUT OF SYSTEMS, TOURS OF VITAL AREAS, PHYSICAL SECURITY, RADIATION PROTECTION, HOUSEKEEPING AND CLEANLINESS, FIRE PROTECTION, OBSERVATION OF MAINTENANCE, OBSERVATION OF SURVEILLANCE AND REVIEW AND FOLLOWUP ON LICENSEE EVENT REPORTS. THIS ROUTINE INSPECTION INVOLVED 50 MANHOURS OF INSPECTION TIME ON SITE IN TEN AREAS IDENTIFIED ABOVE. OF THE TEN AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION APRIL 28 - MAY 1 AND 6-8 (80-17): INCLUDED REVIEW OF SECURITY PLAN; PROTECTED AREA PHYSICAL BARRIERS; VITAL AREA PHYSICAL BARRIERS; LIGHTING; ACCESS CONTROL (IDENTIFICATION, AUTHORIZATION AND BADGING); ACCESS CONTROL (SEARCHING); ACCESS CONTROL (VITAL AREAS); DETECTION AIDS; COMMUNICATIONS; TESTING AND MAINTENANCE; COMPENSATORY MEASURES; POWER SUPPLY; RESPONSE; RECORDS AND REPORTS. THE INSPECTION INVOLVED 40 INSPECTOR-HOURS ON SITE BY TWO NRC INSPECTORS. FIFTEEN INSPECTOR-HOURS WERE ACCOMPLISHED DURING OFFSHIFT PERIODS. OF THE FOURTEEN AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE WERE DISCLOSED IN EIGHT AREAS; SIX APPARENT ITEMS OF NONCOMPLIANCE WERE IDENTIFIED IN THREE AREAS: (1) PHYSICAL BARRIERS - FAILURE TO MAINTAIN ISOLATION ZONE CLEAR OF OBJECTS; (2) PHYSICAL BARRIERS - INOPERATIVE LOCKING DEVICE ON PROTECTED AREA (PA); (3) ACCESS CONTROL - FAILURE TO SEARCH HAND-CARRIED PACKAGES; (4) ACCESS CONTROL - FAILURE TO FOLLOW PROCEDURES FOR VEHICLES ACCESS INTO PA; (5) DETECTION AIDS - INTRUSION ALARMS AT CERTAIN LOCATIONS ALONG THE PA INOPERATIVE; AND (6) DETECTION AIDS - TAMPER INDICATING DEVICES DID NOT PERFORM IN ACCORDANCE WITH SPECIFICATIONS.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

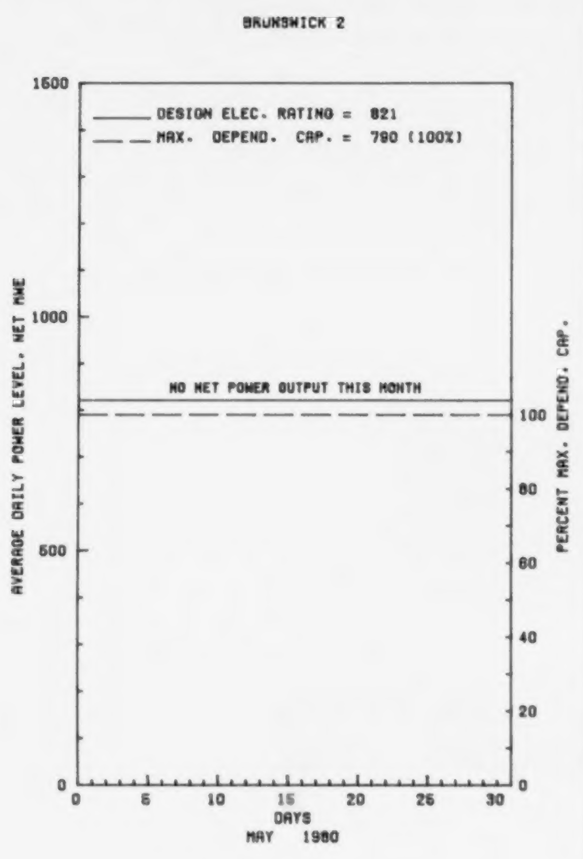
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-24/ 03L-0	03/11/80	04/08/80	BLISTERS FOUND ON TORUS COATING-3 COAT MODEL 6548/7107/7475.
80-25/ 03L-0	03/18/80	04/15/80	CONTROL ROD MOVED DURING REFUELING WITH SHORTING LINK INSTALLED IN REACTOR PROTECTION SYSTEM.
80-27/ 03L-0	03/25/80	04/22/80	LEAK DETECTION LEVEL SWITCH FOR HCU 26-47 INOPERABLE.
80-29/ 03L-0	03/23/80	04/21/80	HPCI TURBINE STOP VALVE OPEN TOO FAST CAUSING HPCI DISCHARGE VALVE TO CYCLE.
80-31/ 01P-0	04/04/80	04/15/80	THRU WALL CRACK IN 3/4 INCH TEST CONNECTION ON REACTOR WATER CLEANUP LINE.

REVIEWED BY OMPA LOEB: Earl Hill **DATE:** 7/3/80

REVIEWED BY NRR DOR: John N. Hannon **DATE:** 7/7/80

BRUNSWICK 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-324
2. Utility Contact: E. Willis (919) 457-9521
3. Licensed Thermal Power (MWt): 2436
4. Nameplate Rating (Gross MWe): 963 x 0.9 = 867
5. Design Electrical Rating (Net MWe): 821
6. Maximum Dependable Capacity (Gross MWe): 815
7. Maximum Dependable Capacity (Net MWe): 790
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	3,647.0	40,128.0
12. Hours Reactor Was Critical	.0	1,365.7	26,689.6
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	.0	1,228.1	25,102.1
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	2,604,960	46,335,422
17. Gross Electrical Energy (MWH)	0	872,613	15,487,687
18. Net Electrical Energy (MWH)	0	842,126	14,882,598
19. Unit Service Factor	.0	33.7	62.6
20. Unit Availability Factor	.0	33.7	62.6
21. Unit Capacity Factor (MDC Net)	.0	29.2	46.9
22. Unit Capacity Factor (DER Net)	.0	28.1	45.2
23. Unit Forced Outage Rate	.0	8.1	12.8

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 06/20/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	03/01/80	S	744.0	C	1		RC	FUELXX	REFUELING OUTAGE CONTINUES.

SUMMARY

THE UNIT REMAINS IN A REFUELING OUTAGE UNTIL 06/20/80.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

CALVERT CLIFFS 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Maryland/Calvert 40 miles S of Annapolis
2.	Electric Reliability Council	Mid-Atlantic Area Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2700
5.	Design Electrical Rating (MWE Net)	845
6.	Date of Initial Criticality	October 7, 1974
7.	Date of Commercial Operation	May 8, 1975
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Chesapeake Bay

Utility & Contractor Information

10.	Licensee	Baltimore Gas and Electric Company
11.	Corporate Address	Gas and Electric Building Baltimore, Maryland 21203
12.	Corporate Contact	A. E. Lundvall, Jr. Vice President-Supply
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Combustion Engineering
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Section Chief	E. C. McCabe, Jr.
19.	IE Resident Inspector	R. Architzel (301) 586-2626
20.	Licensing Project Manager	E. L. Conner, Jr.
21.	Docket Number	50-317
22.	License Number/Date of Issue	DPR-53, 7/31/74
23.	Public Document Room	Calvert County Library Prince Frederick, Maryland 20678

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO T.S. 6.8.1, OI 21 WAS NOT APPROPRIATELY IMPLEMENTED IN THAT APPROPRIATE PRECAUTIONS WERE NOT INCLUDED IN THE SHUTDOWN SECTION TO PREVENT THE SHUTDOWN OF A DIESEL GENERATOR WITH A START SIGNAL PRESENT. (8001 INFRACTION)

CONTRARY TO T.S. 6.5.1.6, VIOLATIONS OF THE T.S. IDENTIFIED IN NRC INSPECTION REPORTS DURING CALENDAR YEAR 1979 WERE NOT INVESTIGATED BY THE PORSC AS REQUIRED AS OF 1/1/80. (8001 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

MANAGERIAL ITEMS:

On July 1, 1980, the Electric Production Department will be split into Fossile Power and Nuclear Power Departments. The following are assignments to supervisory positions in the new Nuclear Power Department.

Mr. Joseph A. Tiernan becomes Department Manager.

Mr. Leon B. Russell, Chief Engineer-Nuclear Plant, becomes Plant Superintendent-Calvert Cliffs.

Mr. Robert E. Denton, Nuclear Plant Engineer-Operations, will be promoted to General Supervisor-Training and Technical Services.

Mr. Walter J. Lippold will continue as Supervisor-Nuclear Fuel Management.

Mr. Norman L. Millis joins the Company as General Supervisor-Radiation Safety.

Mr. Howard A. Eggleston, Assistant General Supervisor-Plant Services at Calvert Cliffs, has been promoted to Supervisor-Administrative Services.

Mr. John T. Carroll, Supervisor-Technical Support at Calvert Cliffs will be promoted to General Supervisor-Operations.

Mr. William S. Gibson, Nuclear Plant Engineer-Performance at Calvert Cliffs becomes General Supervisor-Electrical and Controls.

Mr. Gary E. Brobst will be promoted to Acting General Supervisor-Chemistry. Mr. Brobst presently holds the position of Chemist in the Chemical Unit of the Electric Production Department.

Mr. Richard P. Heibel will be promoted to Acting Supervisor-Technical Support. Mr. Heibel presently holds the position of Engineer in the Operations Unit at Calvert Cliffs.

PLANT STATUS:

No significant new information.

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-005/03L	03/25/80	04/25/80	Unipressure valve diaphragm failure caused trip of #12 control room air conditioner. Valve replacement restored operability in 8.5 hrs.
80-016/03L	03/25/80	04/25/80	"A" wide range NI failed due to 15 VDC pwr supply failure, necessitating replacement with a spare
80-017/03L	03/26/80	04/25/80	ECCS exh filter charcoal inop due inability to remove 90% of methyliodine. The filters were replaced
80-018/03L	04/01/80	04/30/80	No. 11 control rm air conditioner OOS 5hr to repair pneumatic control system (recalibrate outside air temp xmtr)
80-019/03L	03/27/80	04/25/80	Faulty Aux Master Trip Relay caused loss of "B" load trip. The relay was replaced.
80-022/01T	04/15/80	04/25/80	Boron dilution accident with 1% S/D margin & RCS drained to middle of hot leg not analyzed for a 2% S/D margin will be maintained in that plant condition
80-023/03L	04/30/80	05/19/80	Channel B wide range nuclear instrument sur trip inoperable printed circuit board defective, replaced
80-025/04T	04/28/80	05/07/80	Oysters collected in plant environs had levels of radioactivity greater than previous background levels. Attributable to new more sensitive measurements methods

LAST IE SITE INSPECTION DATE: 5/12-23/80

INSPECTION REPORT NO: 50-317/80-09

SUMMARY

INSPECTION SUMMARY

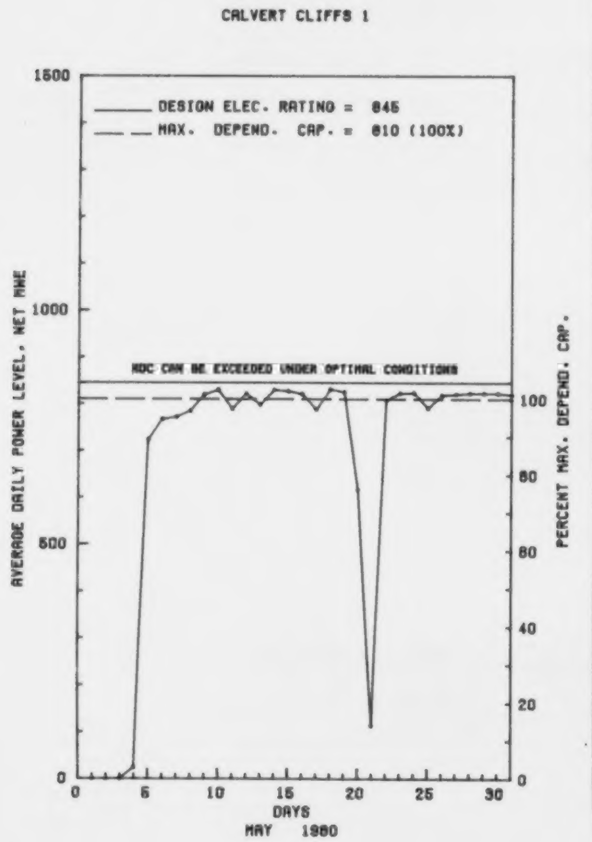
50-317/80-04 - Mar 17-21: Review of Security Plan; Security Program Audit; Security Organization; Protected Area Physical Barriers; Vital Area Physical Barriers; Lighting; Access Control (Identification, Authorization and Badging), Search, Escorting, Vital Areas; Alarm Stations; Detection Aids; Assessment Aids; Communications; Testing and Maintenance; Compensatory Measures; Power Supply; Response; Locks, Keys, and Combinations; Records and Reports; Followup on Previous Items of Noncompliance/Deviations; and Followup on IE Bulletin No. 77-08. The inspection involved 82 hours onsite by 2 region-based inspectors. The inspection began during an offshift period; 18 inspection hours were accomplished during offshift periods. Three items of noncompliance were identified: (Failure to submit the required reports of changes to the accepted Physical Security Plan to the NRC within 10 CFR 50.54(p) specified time limits; Failure to maintain documentation of security force personnel training; and Failure to positively control access to vital areas).

REVIEWED BY OMPA LOEB: L. Tomson **DATE:** 7/3/80

REVIEWED BY NRR DOR: Elon L Conner Jr. **DATE:** 7/7/80

CALVERT CLIFFS 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-317
2. Utility Contact: S. D. Merson (301) 234-5260
3. Licensed Thermal Power (MWt): 2700
4. Nameplate Rating (Gross MWe): 1020 x 0.9 = 918
5. Design Electrical Rating (Net MWe): 845
6. Maximum Dependable Capacity (Gross MWe): 845
7. Maximum Dependable Capacity (Net MWe): 810
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
   NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
   NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	3,647.0	44,412.0
12. Hours Reactor Was Critical	636.1	3,083.0	35,522.6
13. Reactor Reserve Shutdown Hours	19.3	38.3	1,098.8
14. Hours Generator On-Line	629.6	3,013.6	34,715.6
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,648,361	6,795,779	82,132,767
17. Gross Electrical Energy (MWH)	526,927	2,148,392	26,960,922
18. Net Electrical Energy (MWH)	502,362	2,034,231	25,683,437
19. Unit Service Factor	84.6	82.6	78.2
20. Unit Availability Factor	84.6	82.6	78.2
21. Unit Capacity Factor (MDC Net)	83.4	68.9	71.4
22. Unit Capacity Factor (DER Net)	79.9	66.0	68.4
23. Unit Forced Outage Rate	15.4	5.9	8.7

24. Shutdowns Scheduled Over Next 6 Months (Type,Date,and Duration of Each):
   SCHED. OUTAGE 10/17/80 FOR REFUEL, UNIT GEN INSP & TMI MOD.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-08	04/30/80	F	91.2	A	4	LER 80-24/3L	CB	PUMPXX	LEAK ON #11B REACTOR COOLANT PUMP CONTROL BLEED-OFF LINE. PLANT WAS ALREADY OFF FOR OUTAGE (80-07).
80-90	05/20/80	F	23.2	A	1	LER 80-27/1T	PA	HTEXCH	LEAK IN THE AFTER-COOLER ON #12 INSTRUMENT AIR COMPRESSOR.

SUMMARY

THE UNIT RETURNED TO SERVICE ON 05/04/80 FOLLOWING RCS REPAIR AND OPERATED AT FULL POWER EXCEPT FOR REDUCTIONS DUE TO SALTWATER LEAKAGE AND TURBINE VALVE TESTING. A TRIP OCCURRED ON 05/20/80 FOR SERVICE WATER SYSTEM REPAIR.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

CALVERT CLIFFS 2

FACILITY DATA

Facility Description

1. Location (state/countv) (nearest population center)	Maryland/Calvert 40 miles S of Annapolis
2. Electric Reliability Council	Mid-Atlantic Area Council
3. Type of Reactor	PWR
4. Power Level (MWT)	2700
5. Design Electrical Rating (MWE Net)	845
6. Date of Initial Criticality	November 30, 1976
7. Date of Commercial Operation	April 1, 1977
8. Condenser Cooling Method	Once Through
9. Condenser Cooling Water Source	Chesapeake Bay

Utility & Contractor Information

10. Licensee	Baltimore Gas and Electric Company
11. Corporate Address	Gas and Electric Building Baltimore, Maryland 21203
12. Corporate Contact	A. E. Lundvall, Jr. Vice President-Supply
13. Architect/Engineer	Bechtel
14. Nuclear Steam Supply System	Combustion Engineering
15. Constructor	Bechtel

Regulatory Information

16. IE Region responsible	I
17. IE Branch Chief	E. Brunner
18. I E Section Chief	E. C. MCCABE, JR.
19. IE Resident Inspector	R. Architzel (301) 586-2626
20. Licensing Project Manager	E. L. Conner, Jr.
21. Docket Number	50-318
22. License Number/Date of Issue	DPR-69, 8/13/76
23. Public Document Room	Calvert County Library Prince Frederick, Maryland 20678

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO T.S. 6 8.1, OI 21 WAS NOT APPROPRIATELY IMPLEMENTED IN THAT APPROPRIATE PRECAUTIONS WERE NOT INCLUDED IN THE SHUTDOWN SECTION TO PREVENT THE SHUTDOWN OF A DIESEL GENERATOR WITH A START SIGNAL PRESENT. (8001 INFRACTION)

CONTRARY TO T.S. 6.5.1.6, VIOLATIONS OF THE T.S. IDENTIFIED IN NRC INSPECTION REPORTS DURING CALENDAR YEAR 1979 WERE NOT INVESTIGATED BY THE PORSC AS REQUIRED AS OF 1/1/80. (8001 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

MANAGERIAL ITEMS:

On July 1, 1980, the Electric Production Department will be split into Fossil Power and Nuclear Power Departments. The following personnel are assignments to supervisory positions in the new Nuclear Power Department.

Mr. Joseph A. Tiernan becomes Department Manager.

Mr. Leon B. Russel, Chief Engineer-Nuclear Plant, becomes Plant Superintendent-Calvert Cliffs.

Mr. Robert E. Denton, Nuclear Plant Engineer-Operations, will be promoted to General Supervisor-Training and Technical Services.

Mr. Walter J. Lippold will continue as Supervisor-Nuclear Fuel Management.

Mr. Norman L. Millis joins the Company as General Supervisor-Radiation Safety.

Mr. Howard A. Eggleston, Assistant General Supervisor-Plant Services at Calvert Cliffs, has been promoted to Supervisor-Administrative Services.

Mr. John T. Carroll, Supervisor-Technical Support at Calvert Cliffs will be promoted to General Supervisor-Operations.

Mr. William S. Gibson, Nuclear Plant Engineer-Performance at Calvert Cliffs becomes General Supervisor-Electrical and Controls.

Mr. Gary E. Brobst will be promoted to Acting General Supervisor-Chemistry. Mr. Brobst presently holds the position of Chemist in the Chemical Unit of the Electric Production Department.

Mr. Richard P. Heibel will be promoted to Acting Supervisor-Technical Support. Mr. Heibel presently holds the position of Engineer in the Operations Unit at Calvert Cliffs.

PLANT STATUS:

No significant new information.

LAST IE SITE INSPECTION DATE: 5/1-31/80

INSPECTION REPORT NO: 50-318/80-07

NRC OWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-022/ 03L	04/12/80	04/30/80	No. 23 charging pump OOS 3 days to repair suction relief weld leak caused by vibration induced fatigue, piping modifications to eliminate cause sked next refueling outage
80-023/ 03L	04/16/80	05/16/80	Loss of two 15 VDC power output, which supplies power to ESF actuation signal logic B cabinet. The 15 VDC power supplies reset and returned to normal
80-024/ 03L	04/17/80	05/13/80	Reactor coolant pump seal instrument sensing line flexible hose developed a small coolant leak and was replaced. Pump remained operable throughout event
80-025/ 03L	04/28/80	05/16/80	Bypass of Channel A RPS trip units during steady state operation due to failure of Westinghouse P-250 computer open thermocouple detection card
80-026/ 03L	04/28/80	05/16/80	Inadvertent leakage through the pressurizer level control transmitter drain valves during corrective maintenance outage caused erratic pressurizer level indication

SUMMARY

INSPECTION SUMMARY

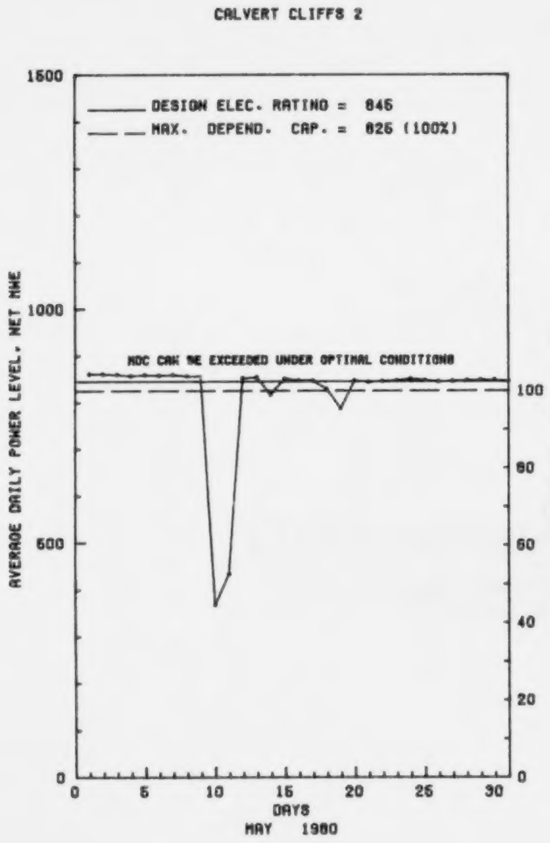
50-318/80-04 - Mar 17-21: Review of Security Plan; Security Program Audit; Security Organization; Protected Area Physical Barriers; Vital Area Physical Barriers; Lighting; Access Control (Identification, Authorization and Badging; Search; Escorting; Vital Areas; Alarm Stations; Detection Aids; Assessment Aids; Communications; Testing and Maintenance; Compensatory Measures; Power Supply; Response; Locks, Keys, and Combinations; Records and Reports; Followup on Previous Items of Noncompliance/Deviations; and Followup on IE Bulletin No. 77-08. The inspection involved 82 hours onsite by 2 region-based inspectors. The inspection began during an offshift period; 18 inspection hours were accomplished during offshift periods. Three items of noncompliance were identified: Failure to submit reports of changes to the Physical Security Plan to the NRC within 10 CFR 50.54(p) specified time limits; Failure to maintain documentation of security force personnel training; and Failure to positively control access to vital areas).

REVIEWED BY OMPA LOEB: L. Tomason **DATE:** 7/3/80

REVIEWED BY NRR DOR: Elen L. Conner, Jr. **DATE:** 7/7/80

CALVERT CLIFFS 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-318
2. Utility Contact: S. D. Merson (301) 234-5240
3. Licensed Thermal Power (MWt): 2700
4. Nameplate Rating (Gross MWe): 1012 x 0.9 = 911
5. Design Electrical Rating (Net MWe): 845
6. Maximum Dependable Capacity (Gross MWe): 860
7. Maximum Dependable Capacity (Net MWe): 825
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

ITEM 21 CALCULATED WITH A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	3,647.0	27,767.0
12. Hours Reactor Was Critical	734.5	3,407.8	23,352.7
13. Reactor Reserve Shutdown Hours	9.5	9.5	422.6
14. Hours Generator On-Line	723.9	3,382.0	22,990.5
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,917,470	8,789,103	57,033,334
17. Gross Electrical Energy (MWH)	636,257	2,949,954	18,955,177
18. Net Electrical Energy (MWH)	609,016	2,822,489	18,079,509
19. Unit Service Factor	97.3	92.7	82.8
20. Unit Availability Factor	97.3	92.7	82.8
21. Unit Capacity Factor (MDC Net)	99.2	93.8	79.8
22. Unit Capacity Factor (DER Net)	96.9	91.6	77.1
23. Unit Forced Outage Rate	2.7	1.1	6.1

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-02	05/10/80	F	20.1	A	1		ZZ	PUMPXX	LOSS OF EXCITATION TO ALL MAIN CIRCULATING WATER PUMPS.

SUMMARY

THE PLANT OPERATED NORMALLY FOR THE REPORTING PERIOD WITH A BRIEF OUTAGE ON 05/10/80 DUE TO WATER PUMP MALFUNCTION.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

NRC POWER RESTRICTIONS: NONE

COOK 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Michigan/Berrien 11 miles S of Benton Harbor
2.	Electric Reliability Council	East Central Area Reliability Coordination Agreement
3.	Type of Reactor	PWR
4.	Power Level (MWT)	3250
5.	Design Electrical Rating (MWE Net)	1054
6.	Date of Initial Criticality	January 18, 1975
7.	Date of Commercial Operation	August 27, 1975
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10.	Licensee	AEP Corporation
11.	Corporate Address	Indiana & Michigan Power Co. 2 Broadway New York, New York 10004
12.	Corporate Contact	John Dolan Senior Executive V. P. Eng.
13.	Architect/Engineer	AEP
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Indiana & Michigan Power Co.

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	D.C. Boyd
19.	IE Resident Inspector	R. Masse (616) 465-6775
20.	Licensing Project Manager	S. Miner
21.	Docket Number	50-315
22.	License Number/Date of Issue	DPR-58 issued 10/25/74
23.	Public Document Room	Maude Preston Palenske Memorial Library 500 Market Street St. Joseph, Michigan 49085

INSPECTION STATUS

ENFORCEMENT STATUS

A VITAL AREA BARRIER WAS FOUND TO BE DEGRADED.
(8005 INFRACTION)

THE LICENSEE FAILED TO IMPLEMENT ADEQUATE BADGE CONTROL.
(8005 DEFICIENCY)

THE LICENSEE FAILED TO CONDUCT PROPER SCREENING IN ACCORDANCE WITH SECURITY PLANS COMMITMENTS.
(8005 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

Reactor shutdown at 9:00 p.m. on May 30, 1980, to begin a scheduled 56 day refueling outage.

LAST IE SITE INSPECTION DATE: April 21-25, 1980

INSPECTION REPORT NO: 80-06

SUMMARY

INSPECTION SUMMARY

Inspection on January 14-18, 21-25, 29-31; February 11-12; March 3-5 and April 15, (80-01): Announced inspection of the licensee's implementation of the Quality Assurance Program in the following areas: QA/QC administration; qualification of personnel; design changes and modifications; records control; receipt, storage and handling; tests and experiments; procurement control; document control; offsite review committee; audits; test and measurement equipment; surveillance and calibration; maintenance; housekeeping and cleanliness; and offsite support staff. The inspection involved 361 inspecter hours onsite by five NRC inspectors. Of the 15 areas inspected, no apparent items of noncompliance or deviations were identified in 9 areas, 6 apparent items of noncompliance were identified in 5 areas (failure to maintain the equipment classification list current; failure to take prompt corrective action on audit findings; failure to establish a formal audit program for the corporate office; failure to follow housekeeping procedures; failure to maintain the master surveillance schedule current; and failure to control lifted leads).

Inspection on January 11 through March 31, (80-04): Operational safety verification including observations, log and records review, ESF system verification, security and condition reports; Licensee Event Reports (LER's); followup on unplanned releases and internal contamination of personnel; and review of small break LOCA procedures and associated documentation and training. The inspection involved 218 inspector hours by the Senior Resident Inspector, Resident Inspector and a regional based operation inspector. Of the areas inspected, two items of noncompliance were identified in one area (Internal Contamination of Personnel). No items of noncompliance or deviations were identified in the other areas inspected.

Inspection on April 21-25, (80-06): Routine, unannounced radiological environmental protection inspection, including a review of program management; a review and discussion of program results, and an observation of groundwater monitoring wells, the onsite absorption pond and various environmental sampling stations. The inspection also included visits to the Berrien County Office of Emergency Preparedness and the Saint Joseph Memorial Hospital to discuss emergency coordination. The inspection involved 32 inspector hours onsite by two NRC inspectors. For the areas inspected, no apparent items of noncompliance or deviations were identified.

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-07/ 03L-0	03/26/80	04/25/80	While revising a surveillance test procedure for the Unit 1 containment spray system it was discovered that some of the valves in the system flow path were not being cycled at 31 day intervals.
80-08/ 03L-0	04/18/80	05/16/80	During the calibration of the containment atmosphere particulate radiation monitor R-11, the detector tube was found to be bad, causing this monitor to be inoperable.
80-09/ 03L-0	04/28/80	05/21/80	While performing spent fuel pit exhaust fan test, STP 028, the charcoal bed exhaust damper would not fully open.

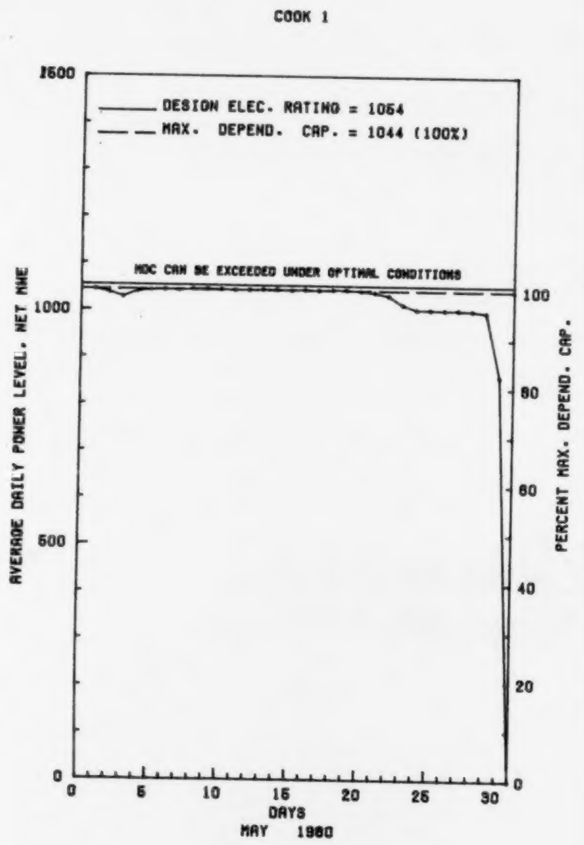
==

REVIEWED BY OMPA LOEB: T. C. Cintula DATE: 7·3·80

REVIEWED BY NRR DOR: Ed Reeves for S. Miner DATE: 8-7-80

COOK 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-315
2. Utility Contact: W. T. Gillett (616) 465-5901
3. Licensed Thermal Power (MWt): 3250
4. Nameplate Rating (Gross MWe): 1280 x 0.9 = 1152
5. Design Electrical Rating (Net MWe): 1054
6. Maximum Dependable Capacity (Gross MWe): 1080
7. Maximum Dependable Capacity (Net MWe): 1044
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	3,647.0	47,471.0
12. Hours Reactor Was Critical	718.5	3,207.9	36,169.6
13. Reactor Reserve Shutdown Hours	.0	.0	463.0
14. Hours Generator On-Line	718.5	3,199.7	35,282.9
15. Unit Reserve Shutdown Hours	.0	.0	321.0
16. Gross Thermal Energy (MWH)	2,305,783	10,118,931	99,185,236
17. Gross Electrical Energy (MWH)	765,850	3,362,810	32,520,150
18. Net Electrical Energy (MWH)	739,608	3,245,802	31,243,116
19. Unit Service Factor	96.6	87.7	78.0
20. Unit Availability Factor	96.6	87.7	78.0
21. Unit Capacity Factor (MDC Net)	95.2	85.2	68.6
22. Unit Capacity Factor (DER Net)	94.3	84.4	64.0
23. Unit Forced Outage Rate	.0	11.6	7.3

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUELING OUTAGE - JUNE & JULY 1980.
25. If Shut Down At End of Report Period, Estimated Date of Startup: 07/25/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
160	05/30/80	S	25.5	C	1		ZZ	ZZZZZZ	THE UNIT WAS REMOVED FROM SERVICE AT 2228 HOURS E.D.T. ON 05/30/80 FOR SCHEDULED CYCLE IV - V REFUELING OUTAGE. THE ESTIMATED OUTAGE TIME IS 61 DAYS.

SUMMARY

THE UNIT RAN ROUTINELY FOR REPORTING PERIOD AND BEGAN REFUELING OUTAGE ON 05/31/80.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

COOK 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Michigan/Berrien 11 miles S of Benton Harbor
2.	Electric Reliability Council	East Central Area Reliability Coordination Agreement
3.	Type of Reactor	PWR
4.	Power Level (MWT)	3391
5.	Design Electrical Rating (MWE Net)	1100
6.	Date of Initial Criticality	March 10, 1978
7.	Date of Commercial Operation	July 1, 1978
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10.	Licensee	AEP Corporation Indiana & Michigan Power Co. Indiana & Michigan Electric Co.
11.	Corporate Address	2 Broadway New York, New York 10004
12.	Corporate Contact	John Dolan, VP/Chief Engineer (212) 422 - 4800
13.	Architect/Engineer	American Elec. Power Service Corp.
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Indiana & Michigan Power Co.

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	D.C. Boyd
19.	IE Resident Inspector	R. Masse (616) 465-6775
20.	Licensing Project Manager	S. Miner
21.	Docket Number	50-316
22.	License Number/Date of Issue	DPR-74, 12/23/77
23.	Public Document Room	Maude Preston Palenske Memorial Library 500 Market Street St. Joseph, Michigan 49085

INSPECTION STATUS

ENFORCEMENT STATUS

A VITAL AREA BARRIER WAS FOUND TO BE DEGRADED.
(8004 INFRACTION)

THE LICENSEE FAILED TO IMPLEMENT ADEQUATE BADGE CONTROL.
(8004 DEFICIENCY)

THE LICENSEE FAILED TO CONDUCT PROPER SCREENING IN ACCORDANCE WITH SECURITY PLANS COMMITMENTS.
(8004 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

Operating routinely.

LAST IE SITE INSPECTION DATE: April 21-25, 1980

INSPECTION REPORT NO: 80-05

SUMMARY

INSPECTION SUMMARY

Inspection on January 14-18, 21-25, 29-31; February 11-12; March 3-5 and April 15, (80-01): Announced inspection of the licensee's implementation of the Quality Assurance Program in the following areas: QA/QC administration; qualification of personnel; design changes and modifications; records control; receipt, storage and handling; tests and experiments; procurement control; document control; offsite review committee; audits; test and measurement equipment; surveillance and calibration; maintenance; housekeeping and cleanliness; and offsite support staff. The inspection involved 361 inspector hours onsite by five NRC inspectors. Of the 15 areas inspected, no apparent items of noncompliance or deviations were identified in 9 areas, 6 apparent items of noncompliance were identified in 5 areas (failure to maintain the equipment classification list current; failure to take prompt corrective action on audit findings; failure to establish a formal audit program for the corporate office; failure to follow housekeeping procedures; failure to maintain the master surveillance schedule current; and failure to control lifted leads).

Inspection on January 11 through March 31, (80-03): Operational safety verification including observations, log and records review, ESF system verification, security and condition reports; Licensee Event Reports (LER's); followup on unplanned releases and internal contamination of personnel; and review of small break LOCA procedures and associated documentation and training. The inspection involved 218 inspector hours by the Senior Resident Inspector, Resident Inspector and a regional based operation inspector. Of the areas inspected, two items of noncompliance were identified in one area (Internal Contamination of Personnel). No items of noncompliance or deviations were identified in the other areas inspected.

Inspection on April 21-25, (80-05): Routine, unannounced radiological environmental protection inspection, including a review of program management; a review and discussion of program results, and an observation of groundwater monitoring wells, the onsite absorption pond and various environmental sampling stations. The inspection also included visits to the Berrien County Office of Emergency Preparedness and the Saint Joseph Memorial Hospital to discuss emergency coordination. The inspection involved 32 inspector hours onsite by two NRC inspectors. For the areas inspected, no apparent items of noncompliance or deviations were identified.

REPORTS RECEIVED FROM LICENSEE

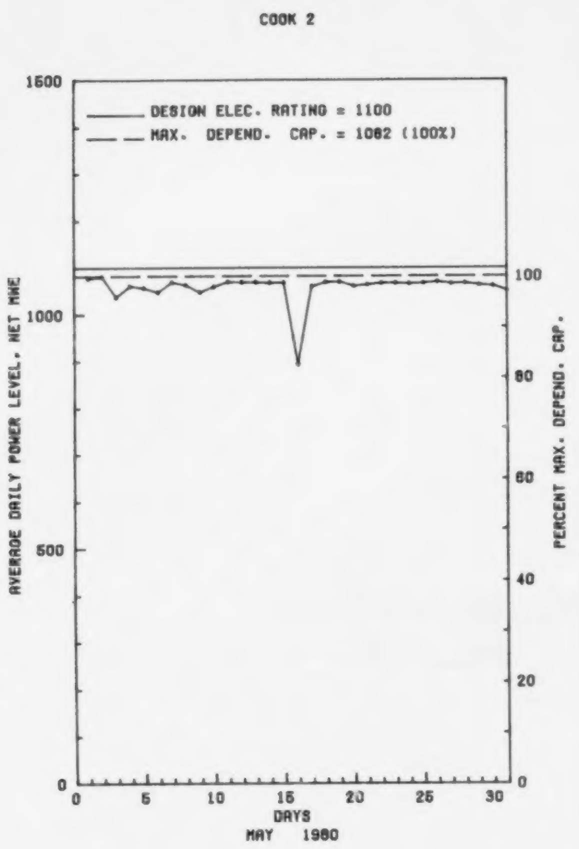
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-07/ 03L-0	01/29/80	02/28/80	During normal operation, while performing a surveillance test, containment isolation valve NCR-252 did not close fully.
80-08/ 03L-0	01/10/80	02/29/80	A review of a surveillance test procedure indicated that the as found data for Loop 1 channel 1, reactor coolant flow transmitter was 3.8% out of calibration.
80-18/ 03L-0	03/31/80	04/25/80	During normal operation, radiation monitor R-11 alarmed and would not reset. Investigation revealed that the sample pump for R-11 and R-12 was not providing sufficient flow.
80-19/ 03L-0	04/11/80	05/06/80	During a rapid power reduction to remove the west feed pump from service because of a faulty coupling, the axial flux difference was forced out of the plus or minus 5% target band defined in TS 3.2.1 for a period of 16 minutes.
80-20 03L-0	04/21/80	05/20/80	During normal operation, it was noticed that steam-line differential pressure between loops 1 and 2 was excessive.
80-21 03L-0	04/29/80	05/20/80	During normal operation, while performing a surveillance test, it was discovered that the bistable for steam generator water level Hi-Hi ESP setpoint (turbine trip and feedwater isolation) had drifted high exceeding the LCO value.

REVIEWED BY OMPA LOEB: T.C. Cintula **DATE:** 7·3·80

REVIEWED BY NRR DOR: Ed Reeves for S. [illegible] **DATE:** 7-7-80

COOK 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-316
2. Utility Contact: W. T. Gillett (616) 465-5901
3. Licensed Thermal Power (MWt): 3391
4. Nameplate Rating (Gross MWe): 1333 x 0.85 = 1133
5. Design Electrical Rating (Net MWe): 1100
6. Maximum Dependable Capacity (Gross MWe): 1118
7. Maximum Dependable Capacity (Net MWe): 1082
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	3,647.0	21,167.0
12. Hours Reactor Was Critical	744.0	3,181.5	14,261.4
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	744.0	3,096.4	13,598.3
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	2,483,250	10,006,416	42,230,796
17. Gross Electrical Energy (MWH)	815,330	3,292,560	13,446,970
18. Net Electrical Energy (MWH)	787,237	3,176,562	12,943,974
19. Unit Service Factor	100.0	84.9	73.0
20. Unit Availability Factor	100.0	84.9	73.0
21. Unit Capacity Factor (MDC Net)	97.8	80.5	67.3
22. Unit Capacity Factor (DER Net)	96.2	79.2	66.2
23. Unit Forced Outage Rate	.0	2.6	11.1

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
77	05/16/80	F	.0	A	5		ZZ	ZZZZZZ	REACTOR POWER REDUCED TO 60% TO REMOVE THE EAST MAIN FEEDPUMP TURBINE FROM SERVICE TO CHECK FOR FEEDPUMP TURBINE CONDENSER TUBE LEAKS. ONE TUBE WAS PLUGGED. REACTOR POWER RETURNED TO 100% 05/17/80.

SUMMARY

THE UNIT WAS RUN ON AN AVERAGE OF 96% POWER LEVEL FOR THE MONTH WITH A BRIEF LOAD REDUCTION ON 05/16/80 FOR REPAIRS TO A FEED PUMP.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG 0161)

[5] Exhibit H:
Same source.

COOPER STATION

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Nebraska/Nemaha 23 miles S of Nebraska City
2.	Electric Reliability Council	Mid-Continent Area Reliability Coordination Agreement
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2381
5.	Design Electrical Rating (MWE Net)	778
6.	Date of Initial Criticality	February 21, 1974
7.	Date of Commercial Operation	July 1, 1974 (50% rating)
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Missouri River

Utility & Contractor Information

10.	Licensee	Nebraska Public Power District
11.	Corporate Address	P. O. Box 499 Columbus, Nebraska 68601
12.	Corporate Contact	J. M. Pilant, Director Licensing and Quality Assurance
13.	Architect/Engineer	Burns and Roe, Inc
14.	Nuclear Steam Supply System	GE
15.	Constructor	Burns and Roe, Inc.

Regulatory Information

16.	IE Region responsible	IV
17.	IE Branch Chief	G. L. Madsen
18.	IE Section Chief	T. F. Westerman
19.	IE Resident Inspector	G. L. Constable
20.	Licensing Project Manager	V. L. Rooney
21.	Docket Number	50-298
22.	License Number/Date of Issue	DPR-46, 1/18/74
23.	Public Document Room	Auburn Public Library 118 15th Street Auburn, Nebraska 68305

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

CRACKS FOUND ON NO. 1 AND 2 LOW PRESSURE TURBINE ROTOR DISC.

FACILITY ITEMS (PLANS AND PROCEDURES):

MANAGERIAL ITEMS:

PLANT STATUS:

SHUT DOWN FOR REFUELING TORUS MODIFICATIONS AND INSPECTION OF WESTINGHOUSE TURBINE FOR CRACKING ON 3/1/80. OUTAGE EXPECTED TO EXTEND TO MAY 31, 1980.

LAST IE SITE INSPECTION DATE: May 6 - 9, 19 - 23, 1980 *

INSPECTION REPORT NO: 50-298/80-08 *

SUMMARY

INSPECTION SUMMARY

Inspection on December 20, 1979 (80-05): As part of the inspection conducted during December 14-21, 1979 at NECO, Richland, Washington, this report details the inspection of trailer 228146. The State of Washington was represented on-site during this inspection. No package was opened. The exterior of the shielding cask was not labeled nor marked to show hazardous materials contained within. This appeared to be in violation with DOT regulation. It appears that the Department of Transportation and the NRC Region IV office are the appropriate organizations to take enforcement actions.

Inspection on April 8-11, 21-23, 29-30, 1980 (80-06): Routine, unannounced inspection of follow-up on previously identified items, quality assurance annual review, calibration, review of in-service inspection procedures, review and evaluation of in-service inspection data, follow-up on IEB 79-01B and independent inspection effort. This inspection involved 105 manhours on-site by four NRC inspectors. Within the areas inspected above no items of noncompliance or deviations were identified.

Inspection on May 6-9, 19-23, 1980 (80-08): Routine, unannounced inspection of follow-up on previously identified items, refueling activities, surveillance, surveillance of pipe supports and restraint systems, maintenance, witness of Containment Integrated Leak Rate Test, and and independent inspection effort. This inspection involved 102 man hours on-site by two NRC inspectors. Within the areas inspected above, no items of noncompliance or deviations were identified.

REPORTS RECEIVED FROM LICENSEE

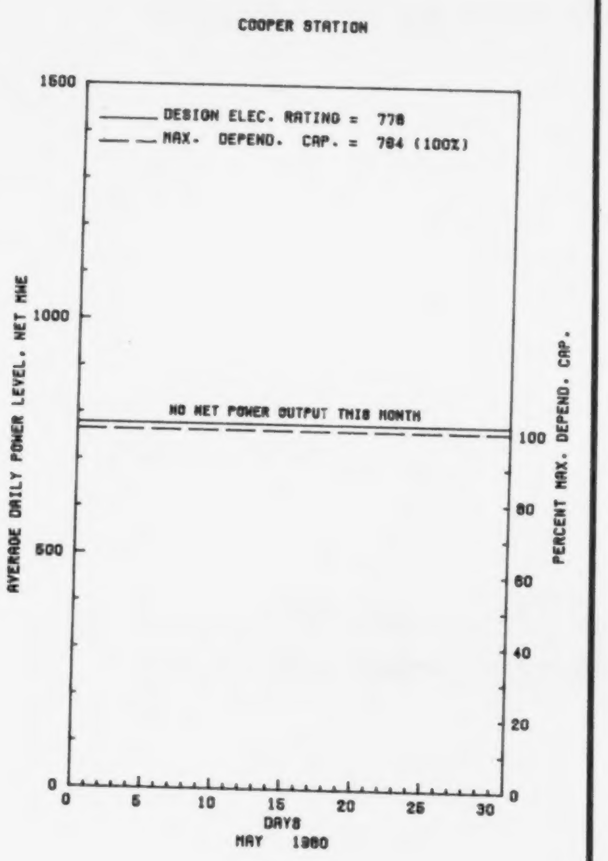
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-09 03L-0	3/29/80	4/28/80	Leak in REC system
80-10 03L-0	4/8/80	5/8/80	Repair of core spray pipe weld
80-11 03L-0	4/18/80	5/12/80	Failure of BUNA-N seats in ADS air supply check valves

REVIEWED BY OMPA LOEB: **DATE:** 7/3/80

REVIEWED BY NRR DOR: **DATE:** 7/7/80

COOPER STATION

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-298
2. Utility Contact: P. J. Borer (402) 825-3811
3. Licensed Thermal Power (MWt): 2381
4. Nameplate Rating (Gross MWe): 983 x 0.85 = 836
5. Design Electrical Rating (Net MWe): 778
6. Maximum Dependable Capacity (Gross MWe): 787
7. Maximum Dependable Capacity (Net MWe): 764
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	3,647.0	51,888.0
12. Hours Reactor Was Critical	.0	1,447.0	42,684.3
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	.0	1,437.7	41,913.3
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	3,077,520	79,260,246
17. Gross Electrical Energy (MWH)	0	1,018,195	25,557,063
18. Net Electrical Energy (MWH)	0	981,128	24,639,405
19. Unit Service Factor	.0	39.4	80.8
20. Unit Availability Factor	.0	39.4	80.8
21. Unit Capacity Factor (MDC Net)	.0	35.2	62.2
22. Unit Capacity Factor (DER Net)	.0	34.6	61.0
23. Unit Forced Outage Rate	.0	1.7	4.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUELING (03/01/80 TO 06/03/80)
25. If Shut Down At End of Report Period, Estimated Date of Startup: 05/04/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-9	03/01/80	S	744.0	C	4		RC	FUELXX	CONTINUATION OF THE REFUELING OUTAGE BEGUN ON 03/01/80.

SUMMARY

THE UNIT WAS SHUTDOWN THROUGHOUT THE REPORTING PERIOD FOR REFUELING AND MAINTENANCE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

CRYSTAL RIVER 3

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Florida/Citrus 7 miles NW of Crystal River
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2452
5.	Design Electrical Rating (MWE Net)	825
6.	Date of Initial Criticality	January 14, 1977
7.	Date of Commercial Operation	March 13, 1977
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Gulf of Mexico

Utility & Contractor Information

10.	Licensee	Florida Power Corporation
11.	Corporate Address	3201 34th Street, South St. Petersburg, Florida 33733
12.	Corporate Contact	J. Hancock
13.	Architect/Engineer	Gilbert Associates, Inc.
14.	Nuclear Steam Supply System	B&W
15.	Constructor	J. A. Jones Construction Co.

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. Lewis
18.	IE Section Chief	R. Martin
19.	IE Resident Inspector	T. F. Stetka
20.	Licensing Project Manager	P. B. Erickson
21.	Docket Number	50-302
22.	License Number/Date of Issue	DPR-72, 12/3/76
23.	Public Document Room	Crystal River Public Library 668 N.W. First Crystal River, Florida 32629

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO 10 CFR 50, APPENDIX B, CRITERION VI, MEASURES HAD NOT BEEN ESTABLISHED TO ASSURE THAT ALL APPROVED CHANGES TO VENDOR MANUALS ARE PROMPTLY ENTERED AND THAT OBSOLETE OR SUPERSEDED DOCUMENTS ARE REMOVED FROM USE.
(8019 INFRACTION)

CONTRARY TO 10 CFR 50, APPENDIX B, CRITERION XVII, QUALITY ASSURANCE RECORDS WERE NOT ALWAYS DISTRIBUTED AND HANDLED IN ACCORDANCE WITH WRITTEN PROCEDURES AND/OR THE STORAGE SYSTEMS DID NOT ALWAYS PROVIDE FOR THE ACCURATE RETRIEVAL OF INFORMATION WITHOUT UNDUE DELAY.
(8019 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

ANTICIPATED CRITICALITY FOLLOWING OUTAGE 7/14/80.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

REFUELING SHUTDOWN.

LAST IE SITE INSPECTION DATE: MAY 5-30, 1980

INSPECTION REPORT NO: 50-302/80-23

SUMMARY

INSPECTION SUMMARY

INSPECTION MARCH 31 - APRIL 11 (80-18): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 88 INSPECTOR-HOURS ON SITE IN THE AREAS OF FOLLOWUP ON BULLETIN 79-01B, FOLLOWUP ON PREVIOUS INSPECTION FINDINGS, FUEL RECEIPT DOCUMENTATION, WITNESSING OF REFUELING ACTIVITIES, AND MONITORING OF OUTAGE MAINTENANCE ACTIVITIES. OF THE FIVE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION APRIL 21-25 (80-19): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 33 INSPECTOR-HOURS ON SITE IN THE AREAS OF RECORDS PROGRAM, DOCUMENT CONTROL PROGRAM AND PLANT TOUR. OF THE THREE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN ONE AREA; TWO APPARENT ITEMS OF NONCOMPLIANCE WERE FOUND IN TWO AREAS (INFRACTION - FAILURE TO PROPERLY IMPLEMENT RECORD CONTROL PROGRAM; AND INFRACTION - FAILURE TO PROVIDE ADEQUATE DOCUMENT CONTROL PROGRAM FOR VENDOR MANUALS).

INSPECTION APRIL 14-19 (80-20): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 83 INSPECTOR-HOURS ON SITE IN THE AREAS OF REFUELING, FOLLOWUP ON LICENSEE IDENTIFIED ITEMS, AND TOURING THE PLANT. OF THE THREE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION APRIL 23-25 (80-21): THIS SPECIAL, ANNOUNCED INSPECTION INVOLVED 17 INSPECTOR-HOURS ON SITE IN THE AREAS OF INSTALLED EQUIPMENT REVIEW WITH RESPECT TO IE BULLETIN 79-01B RESPONSE. OF THE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION APRIL 28 - MAY 2 (80-22): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 31 INSPECTOR-HOURS ON SITE. THE AREAS INSPECTED INCLUDED REFUELING AND RELATED SURVEILLANCE ACTIVITIES AND LICENSEE RESPONSE TO IE BULLETINS. NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

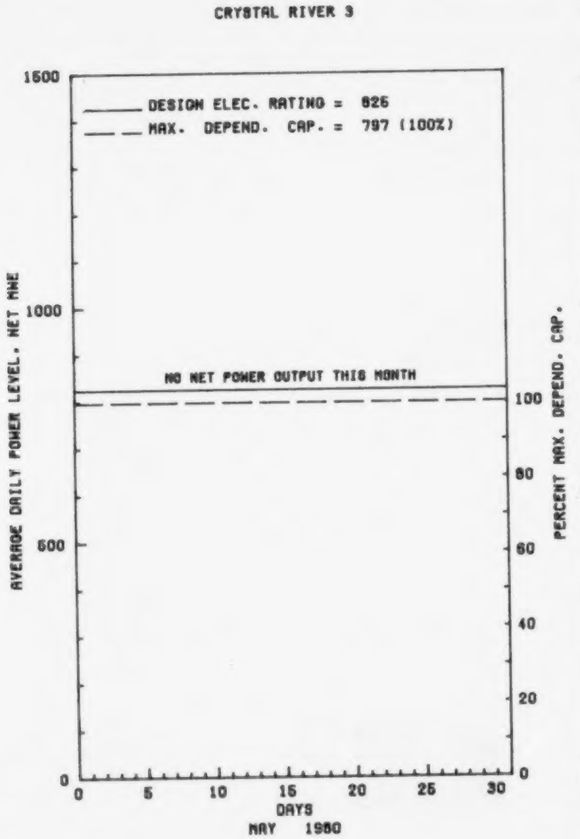
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-016/01T-0	04/01/80	04/16/80	INADVERTENT ACTUATION OF THE FIRE SERVICE DELUGE SYSTEM.
80-017/03L-0	04/05/80	05/01/80	B OTSG FEEDWATER SWING CHECK VALVE FWV-46 INOP DUE TO LOST RETAINER PIN PREVENTING DISK FROM CLOSING CONT ISO VALVE INOP IS CONTRARY TO 3.6.3.1 MODE 5 OPERATION.

REVIEWED BY OMPA LOEB: Bobe **DATE:** 7/3/80

REVIEWED BY NRR DOR: M. B. Fairtile for ERICKSON **DATE:** 7/7/80

CRYSTAL RIVER 3

AVERAGE POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-302
2. Utility Contact: M.W. CULVER (904) 795-6486
3. Licensed Thermal Power (MWt): 2452
4. Nameplate Rating (Gross MWe): 989 x 0.9 = 890
5. Design Electrical Rating (Net MWe): 825
6. Maximum Dependable Capacity (Gross MWe): 821
7. Maximum Dependable Capacity (Net MWe): 797
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	3,647.0	28,223.0
12. Hours Reactor Was Critical	.0	1,305.5	16,455.7
13. Reactor Reserve Shutdown Hours	.0	.0	1,116.9
14. Hours Generator On-Line	.0	1,297.4	15,997.7
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	3,085,927	35,402,838
17. Gross Electrical Energy (MWH)	0	1,058,396	12,030,165
18. Net Electrical Energy (MWH)	0	1,009,335	11,400,975
19. Unit Service Factor	.0	35.6	56.7
20. Unit Availability Factor	.0	35.6	56.7
21. Unit Capacity Factor (MDC Net)	.0	34.7	50.7
22. Unit Capacity Factor (DER Net)	.0	33.[illegible]	49.0
23. Unit Forced Outage Rate	.0	18.0	32.0

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
CONTINUING REFUELING OUTAGE.
25. If Shut Down At End of Report Period, Estimated Date of Startup: 07/13/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-9	03/07/80	S	744.0	C	4		ZZ	ZZZZZZ	CONTINUATION OF THE REFUELING OUTAGE BEGUN ON 03/07/80.

SUMMARY

THE UNIT CONTINUED ANNUAL REFUELING OUTAGE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

DAVIS-BESSE 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Ohio/Ottawa 21 miles E of Toledo
2.	Electric Reliability Council	East Central Area Reliability Coordination Agreement
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2772
5.	Design Electrical Rating (MWE Net)	906
6.	Date of Initial Criticality	August 12, 1977
7.	Date of Commercial Operation	11/21/77 (25%); 12/19/77 (40%) 01/23/78 (75%); 07/31/78 (100%)
8.	Condenser Cooling Method	Cooling Tower
9.	Condenser Cooling Water Source	Lake Erie

Utility & Contractor Information

10.	Licensee	Toledo Edison Co.
11.	Corporate Address	Edison Plaza, 300 Madison Avenue Toledo, Ohio 43652
12.	Corporate Contact	R. P. Crouse, V. P., Nuclear (419)259-5221
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	B&W
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	T. N. TAMBLING
19.	IE Resident Inspector	L. Reyes (419) 898-2775
20.	Licensing Project Manager	D. Garner
21.	Docket Number	50-346
22.	License Number/Date of Issue	NPF-3, 4/22/77
23.	Public Document Room	Ida Rupp Public Library 310 Madison Street Port Clinton, OH 43452

INSPECTION STATUS

ENFORCEMENT STATUS

QUALITY ASSURANCE PROCEDURE 2180 STATES AS A MINIMUM, EACH AFFECTED EXTERNAL AND INTERNAL ORGANIZATION SHALL BE AUDITED FOR COMPLIANCE WITH THE APPROPRIATE 10 CFR 50, APPENDIX B CRITERIA, NRC REGULATORY GUIDES AND ANSI STANDARDS AT LEAST ANNUALLY, OR ONCE DURING THE LIFETIME OF THE ACTIVITY, WHICHEVER IS SHORTER. CONTRARY TO THE ABOVE, THE TECHNICAL SECTION WAS NOT AUDITED SPECIFICALLY FOR NUCLEAR FUEL MANAGEMENT ACTIVITIES, DURING THE CALENDAR YEAR 1978.
(8008 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS

Fuel assembly holddown spring broken. (80-10)

PLANT STATUS

The refueling and maintenance outage continued during the month. The fuel shuffle was completed, however, on May 13, 1980 during an incore inspection a broken fuel assembly holddown spring was found. In a more detailed inspection a total of 11 broken springs were identified out of 112 fuel assemblies inspected. The licensee and B&W are investigating the causes and corrective actions.

LAST IE SITE INSPECTION DATE: April 30 through May 2, 1980

INSPECTION REPORT NO: 80-13

SUMMARY

INSPECTION SUMMARY

Inspection on March 10-14, 17-21, 24-28, 30-31; and April 1-4, (80-06): Routine, unannounced inspection of followup on previous inspection findings; Operational Safety Information Meeting with local officials, CILRT Procedure, Radiation Protection, reactor trip on March 27 and April 7, 1980, followup of Licensee Event Reports and IE Bulletin followup. The inspection involved 166 inspector hours onsite by the Resident Inspector. No items of noncompliance or deviations were identified.

Inspection on April 15-17, 29-30; May 1, 6-8 and 12-14, (80-10): Routine, unannounced inspection of preparation for a refueling outage, pre-fuel handling activities, fuel handling activities, maintenance during a refueling outage, review of previous inspection findings, review of procedure for PG-PL Woodward Governors, and followup on a licensee identified event. The inspection involved 88 inspector hours onsite by two NRC inspectors and included inspection effort conducted during offshift hours. No items of noncompliance or deviations were identified.

Inspection on April 30 through May 2, (80-13): Included a review of Protected Area Physical Barriers; Vital Area Physical Barriers; Access Control (Identification, Authorization and Badging); Access Control (Search); Access Control (Escorting); Access Control (Vital Areas); Testing and Maintenance; Followup on Previous Items of Noncompliance. The inspection involved 66 inspector hours onsite by 3 NRC inspectors. The inspection began during day shift period; 18 inspector hours were accomplished during offshift periods. The plant is currently in a refueling outage. The licensee was found to be in compliance with NRC requirements within the seven areas examined during this inspection except for the following five items: Access Control (Identification, Authorization and Badging) - The licensee did not conduct proper personnel screening in accordance with security plan commitments; Access Control (Vital Areas) - A vital area door was found unsecured and unattended and several watchmen were not provided with adequate communication; Access Control (Search) - The licensee failed to adequately search some individuals prior to their entry into the protected area; Access Control (Escorting) - The licensee failed to properly escort several individuals; and Testing and Maintenance - The licensee failed to perform a required test of an alarm system.

REPORTS RECEIVED FROM LICENSEE

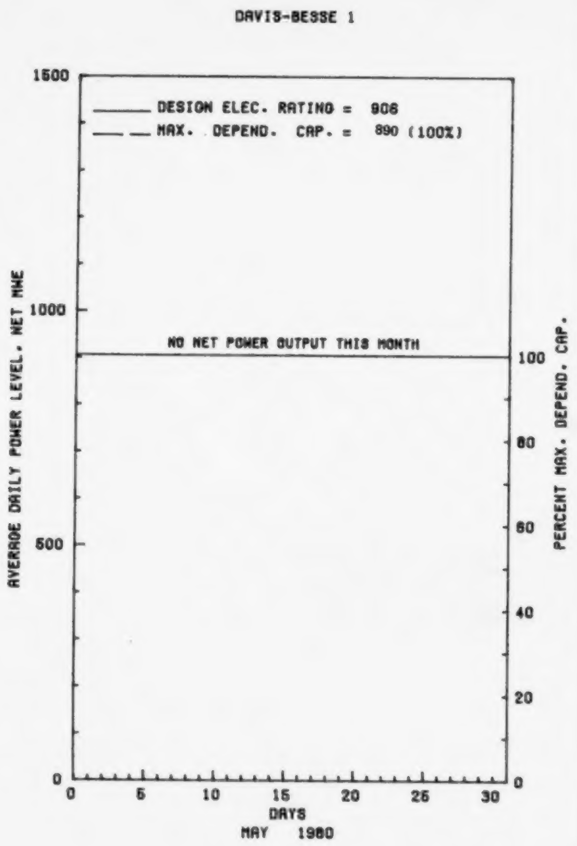
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-23/ 03L-0	03/27/80	04/24/80	Control rods group 3 and 4 were moving in on their own.
80-24/ 03L-0	03/27/80	04/24/80	AFPT main steam isolation valve closed.
80-25/ 03L-0	03/30/80	04/25/80	Control rod group absolute position indication declared inoperable.
80-26/ 03L-0	03/30/80	04/25/80	Smoke detector went into alarm and would not clear.
80-27/ 03L-0	03/31/80	04/28/80	Operations received a half trip of steam and feedwater rupture control system.
80-29/ 03L-0	04/19/80	04/30/80	Breaker on "B" bus opened.
80-30/ 01T-0	04/18/80	05/01/80	Decay heat pump stopped when RCS water level decreased.
80-31/ 03L-0	04/06/80	05/02/80	Weekly test missed it late date.
80-32/ 03L-0	04/06/80	05/02/80	Inboard bearing oil sight glass of CSP shattered.
80-33/ 01T-0	04/25/80	05/06/80	Hydraulic snubber deficiencies.
80-34/ 03L-0	04/11/80	05/07/80	AFP turbine steam inlet press. switch actuated above its setpoint.
80-35/ 03L-0	04/10/80	05/07/80	Decay Heat Removal Cooler outlet flow control valve would not open.
80-36/ 03L-0	04/07/80	05/06/80	Channel trip module only reflected trip signals from 3 of 4 RPS channels.

REVIEWED BY OMPA LOEB: E Bobe **DATE:** 7/3/80

REVIEWED BY NRR DOR: DJ Garner **DATE:** 7/2/80

DAVIS-BESSE 1

AVERAGE POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-346
2. Utility Contact: BILAL SARSOUR (419) 259-5000 X236
3. Licensed Thermal Power (MWt): 2772
4. Nameplate Rating (Gross MWe): 1069 x 0.9 = 962
5. Design Electrical Rating (Net MWe): 906
6. Maximum Dependable Capacity (Gross MWe): 934
7. Maximum Dependable Capacity (Net MWe): 890
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	3,647.0	24,172.0
12. Hours Reactor Was Critical	.0	2,078.2	13,042.4
13. Reactor Reserve Shutdown Hours	.0	.0	2,875.8
14. Hours Generator On-Line	.0	2,008.7	11,883.4
15. Unit Reserve Shutdown Hours	.0	.0	1,728.2
16. Gross Thermal Energy (MWH)	0	4,687,304	24,886,811
17. Gross Electrical Energy (MWH)	0	1,583,559	8,307,070
18. Net Electrical Energy (MWH)	0	1,483,787	7,654,365
19. Unit Service Factor	.0	55.1	50.4
20. Unit Availability Factor	.0	55.1	58.2
21. Unit Capacity Factor (MDC Net)	.0	45.7	38.3
22. Unit Capacity Factor (DER Net)	.0	44.9	37.6
23. Unit Forced Outage Rate	.0	14.3	25.6

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 08/01/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
4	04/07/80	S	744.0	C	4		RC	FUELXX	UNIT OUTAGE WHICH BEGAN ON APRIL 7, 1980 WAS STILL IN PROGRESS THROUGH THE END OF MAY, 1980.

SUMMARY

THE UNIT REMAINED SHUTDOWN FOR REFUELING AND MAJOR MODIFICATIONS ORDERED BY NUREG 0578 (TMI LESSONS LEARNED) THROUGHOUT MAY.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

DRESDEN 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Illinois/Grundy 9 miles E of Morris
2.	Electric Reliability Council	Mid-America Interpool Network
3.	Type of Reactor	BWR
4.	Power Level (MWT)	700
5.	Design Electrical Rating (MWE Net)	200
6.	Date of Initial Criticality	October 15, 1959
7.	Date of Commercial Operation	July 4, 1960
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Kankakee River

Utility & Contractor Information

10.	Licensee	Commonwealth Edison Co.
11.	Corporate Address	P. O. Box 767 Chicago, Illinois 60690
12.	Corporate Contact	C. Reed
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	GE
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	R. Spessard
19.	IE Resident Inspector	J. L. Barker (815)-942-9267
20.	Licensing Project Manager	Paul W. O'Connor
21.	Docket Number	50-010
22.	License Number/Date of Issue	DPR-2 issued 9/28/59
23.	Public Document Room	Morris Public Library 604 Liberty Street Morris, Illinois 60451

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

Unit 1 shutdown on October 31, 1978 for an extended outage to upgrade ECCS, chemical cleaning and refueling. Outage expected to last a minimum of 30 months.

LAST IE SITE INSPECTION DATE: April 7 through May 2, 1980

INSPECTION REPORT NO: 80-08

SUMMARY

INSPECTION SUMMARY

Inspection on March 3-5, 7, 13-14 and April 8-9, (80-08): Routine, announced inspection of radiation protection program for the Unit 1 decontamination, including: audits; training; ALARA; monitoring equipment and procedures; exposure control (external and internal); posting, labeling and control; and notification of reports. The inspection involved 73 inspector hours onsite by one NRC inspector. Of the seven areas inspected, two items of noncompliance were found in one area (failure to make an adequate evaluation to determine compliance with 10 CFR 20.106 and concentrations in excess of TS 3.8.C.2 limits were found in the discharge canal).

Inspection on April 7 through May 2, (80-08): Routine, unannounced resident inspection of operational safety verification, maintenance observation, routine surveillance observation, and inspection during long term shutdown. The inspection involved 123 inspector hours onsite by two NRC inspectors. Of the six areas inspected, there were no items of noncompliance identified in five areas. There were three items of noncompliance (failure to follow procedures; failure to adher to QA program requirements; and failure to follow procedures) in one area.

REPORTS RECEIVED FROM LICENSEE

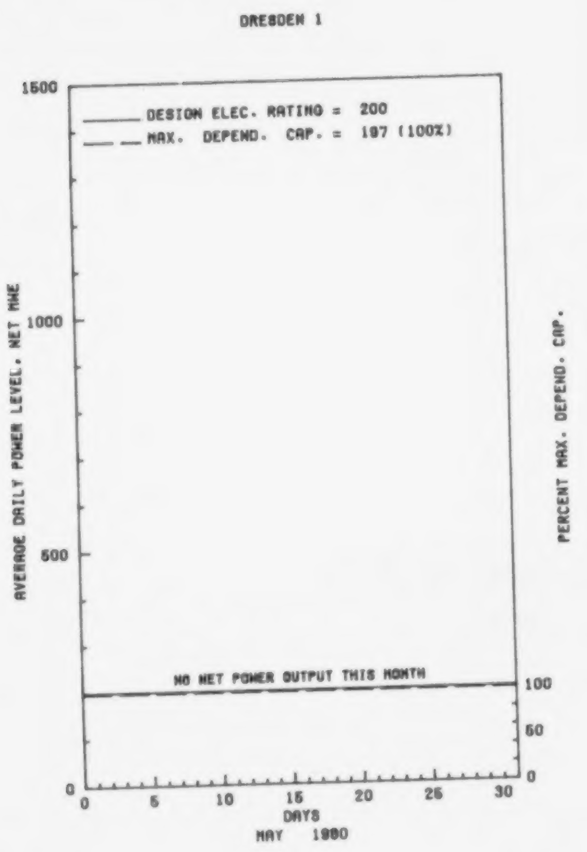
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-02/01T-0	04/05/80	04/17/80	During shutdown mode with fuel removed contaminated demineralized water was heated and added to primary system through the unloader heat exchangers.

REVIEWED BY OMPA LOEB: ______ DATE: 7/3/80

REVIEWED BY NRR DOR: ______ DATE: 7/8/80

DRESDEN 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-010
2. Utility Contact: S. A. Cieszkiewicz (815) 942-2920
3. Licensed Thermal Power (MWt): 700
4. Nameplate Rating (Gross MWe): 245.5 x 0.85 = 209
5. Design Electrical Rating (Net MWe): 200
6. Maximum Dependable Capacity (Gross MWe): 205
7. Maximum Dependable Capacity (Net MWe): 197
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	3,647.0	176,447.0
12. Hours Reactor Was Critical	.0	.0	113,153.9
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	.0	.0	111,196.2
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	0	58,014,747
17. Gross Electrical Energy (MWH)	0	0	16,793,254
18. Net Electrical Energy (MWH)	-1,394	-4,709	15,805,118
19. Unit Service Factor	.0	.0	63.2
20. Unit Availability Factor	.0	.0	63.0
21. Unit Capacity Factor (MDC Net)	.0	.0	45.5
22. Unit Capacity Factor (DER Net)	.0	.0	44.8
23. Unit Forced Outage Rate	.0	.0	11.2

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: JULY 1981

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
8	10/31/78	S	744.0	H	4		RC	FUELXX	REFUEL AND CHEMICAL CLEANING OUTAGE.

SUMMARY

THE UNIT WAS SHUTDOWN FOR REPORTING PERIOD FOR REFUELING AND CHEMICAL CLEANING.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

NRC POWER RESTRICTIONS: NONE

DRESDEN 2

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Illinois/Grundy 9 miles E of Morris
2.	Electric Reliability Council	Mid-America Interpool Network
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2527
5.	Design Electrical Rating (MWE Net)	794
6.	Date of Initial Criticality	January 7, 1970
7.	Date of Commercial Operation	JULY, 1970
8.	Condenser Cooling Method	Cooling Lake
9.	Condenser Cooling Water Source	Kankakee and Des Plaines Rivers

Utility & Contractor Information

10.	Licensee	Commonwealth Edison Co.
11.	Corporate Address	P. O. Box 767 Chicago, Illinois 60690
12.	Corporate Contact	C. Reed
13.	Architect/Engineer	Sargent & Lundy
14.	Nuclear Steam Supply System	GE
15.	Constructor	United Engineers & Constructors, Inc.

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	R. Spessard
19.	IE Resident Inspector	J. L. Barker (815)-942-9267
20.	Licensing Project Manager	Paul W. O'Connor
21.	Docket Number	50-237
22.	License Number/Date of Issue	DPR-19 issued 12/22/69
23.	Public Document Room	Morris Public Library 604 Liberty Street Morris, Illinois 60451

INSPECTION STATUS

ENFORCEMENT STATUS

UNIT 2 AND UNIT 3 TECHNICAL SPECIFICATIONS, SECTION 6.2.B REQUIRE THAT RADIATION CONTROL PROCEDURES BE MAINTAINED, MADE AVAILABLE TO ALL STATION PERSONNEL, AND ADHERED TO. STATION RADIATION CONTROL PROCEDURE 37-1-A-1, REQUIRES THAT EACH PERSON READ, UNDERSTAND, AND ADHERE TO THE PROVISIONS OF SPECIAL WORK PERMITS (SWP'S). CONTRARY TO THE ABOVE, DURING ROUTINE TOURS OF THE UNIT 2 CORE SPRAY/LPCI CORNER ROOMS ON MARCH 6, 1980, AND THE UNIT 2/3 REFUELING FLOOR ON MARCH 28, 1980, THE INSPECTOR OBSERVED PERSONNEL IN CONTROLLED, CONTAMINATED AREAS NOT WEARING ALL OF THE PROTECTIVE CLOTHING REQUIRED BY THE APPLICABLE SWP. (8007 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

Unit operating routinely.

LAST IE SITE INSPECTION DATE: April 7 through May 2, 1980

INSPECTION REPORT NO: 80-09

SUMMARY

INSPECTION SUMMARY

Inspection on April 7 through May 2, (80-09): Routine, unannounced resident inspection of operational safety verification, maintenance observation, and routine surveillance observation. The inspection involved 123 inspector hours onsite by two NRC inspectors. Of the six areas inspected, there were no items of noncompliance identified.

REPORTS RECEIVED FROM LICENSEE

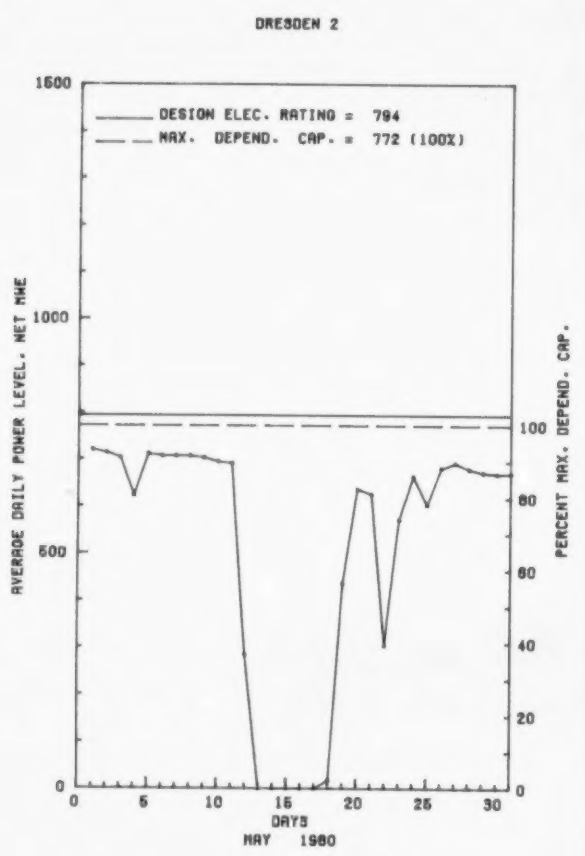
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
NONE				

REVIEWED BY OMPA LOEB: DATE: 7/5/80

REVIEWED BY NRR DOR: DATE: 7/5/80

DRESDEN 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-237
2. Utility Contact: S. A. Cieszkiewicz (815) 942-2920 x263
3. Licensed Thermal Powe. (MWt): 2527
4. Nameplate Rating (Gross MWe): 920 x 0.9 = 828
5. Design Electrical Rating (Net MWe): 794
6. Maximum Dependable Capacity (Gross MWe): 812
7. Maximum Dependable Capacity (Net MWe): 772
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Powe. Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	3,647.0	88,103.0
12. Hours Reactor Was Critical	599.3	3,487.0	68,223.7
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	578.6	3,456.6	64,719.3
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,287,238	8,113,608	129,755,440
17. Gross Electrical Energy (MWH)	401,639	2,582,138	41,588,759
18. Net Electrical Energy (MWH)	378,731	2,452,499	39,340,611
19. Unit Service Factor	77.8	94.8	73.5
20. Unit Availability Factor	77.8	94.8	73.5
21. Unit Capacity Factor (MDC Net)	65.9	87.1	57.8
22. Unit Capacity Factor (DER Net)	64.1	84.7	56.2
23. Unit Forced Outage Rate	3.5	1.3	12.8

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
3	05/12/80	F	10.0	G	3				RX SCRAMMED ON MSL HI RAD (SPURIOUS SIGNAL).
4	05/12/80	S	144.6	A	1				MAINTENANCE OUTAGE - REPLACED "B" RECIRC. PUMP SEALS.
5	05/22/80	F	10.8	G	3				RX SCRAMMED ON RX LOW LEVEL (INSTRUMENT RACK JARRED).

SUMMARY

THE UNIT RAN AT 720 MWE UNTIL 05/12/80, AT WHICH TIME IT WAS SHUTDOWN FOR MAINTENANCE OF "B" RECIRC. PUMP SEALS. ON 05/18/80, IT WAS RETURNED TO SERVICE AND RUN AT AVERAGE POWER LEVEL AT 821 MWE, FOR THE REST OF THE MONTH WITH A UNIT SCRAM ON 05/22/80 DUE TO LOW REACTOR WATER LEVEL.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

NRC POWER RESTRICTIONS: NONE

DRESDEN 3

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Illinois/Grundy 9 miles E of Morris
2.	Electric Reliability Council	Mid-America Interpool Network
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2527
5.	Design Electrical Rating (MWE Net)	794
6.	Date of Initial Criticality	January 31, 1971
7.	Date of Commercial Operation	November 16, 1971
8.	Condenser Cooling Method	Cooling Lake
9.	Condenser Cooling Water Source	Kankakee and Des Plaines Rivers

Utility & Contractor Information

10.	Licensee	Commonwealth Edison Co.
11.	Corporate Address	P. O. Box 767 Chicago, Illinois 60690
12.	Corporate Contact	D. L. Peoples
13.	Architect/Engineer	Sargent & Lundy
14.	Nuclear Steam Supply System	GE
15.	Constructor	United Engineers & Constructors, Inc.

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	R. Spessard
19.	IE Resident Inspector	J. L. Barker 815-942-9267
20.	Licensing Project Manager	R. Bevan
21.	Docket Number	50-249
22.	License Number/Date of Issue	DPR-25 issued 1/12/71
23.	Public Document Room	Morris Public Library 604 Liberty Street Morris, Illinois 60451

INSPECTION STATUS

ENFORCEMENT STATUS

UNIT 2 AND UNIT 3 TECHNICAL SPECIFICATIONS, SECTION 6.2.B REQUIRE THAT RADIATION CONTROL PROCEDURES BE MAINTAINED, MADE AVAILABLE TO ALL STATION PERSONNEL, AND ADHERED TO. STATION RADIATION CONTROL PROCEDURE 37-1-A-1, REQUIRES THAT EACH PERSON READ, UNDERSTAND, AND ADHERE TO THE PROVISIONS OF SPECIAL WORK PERMITS (SWP'S). CONTRARY TO THE ABOVE, DURING ROUTINE TOURS OF THE UNIT 2 CORE SPRAY/LPCI CORNER ROOMS ON MARCH 6, 1980, AND THE UNIT 2/3 REFUELING FLOOR ON MARCH 28, 1980, THE INSPECTOR OBSERVED PERSONNEL IN CONTROLLED, CONTAMINATED AREAS NOT WEARING ALL OF THE PROTECTIVE CLOTHING REQUIRED BY THE APPLICABLE SWP. (8010 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

Operating routinely.

LAST IE SITE INSPECTION DATE: April 7 through May 2, 1980

INSPECTION REPORT NO: 80-12

SUMMARY

INSPECTION SUMMARY

Inspection on April 10-11, (80-09): Reactor Water Clean Up (RWCU) pipe system damage due to water hammer, seismic restraint provisions for the Control Rod Drive (CRD) systems. The inspection involved 10 inspector hours onsite by one NRC inspector. No items of noncompliance or deviations were identified.

Inspection on April 15-18, (80-11): Routine, unannounced inspection of operational radiation protection program during refueling and major maintenance, including: audits; training; planning and preparation; ALARA; exposure control; posting, labeling, and control; and outage activities. The inspection involved 31 inspector hours onsite by one NRC inspector. No items of noncompliance or deviations were identified.

Inspection on April 7 through May 2, (80-12): Routine, unannounced resident inspection of operational safety verification, maintenance observation, routine surveillance observation, startup testing-refueling, major surveillance-outage. The inspection involved 123 inspector hours onsite by two NRC inspectors. Of the six areas inspected, there were no items of noncompliance identified.

REPORTS RECEIVED FROM LICENSEE

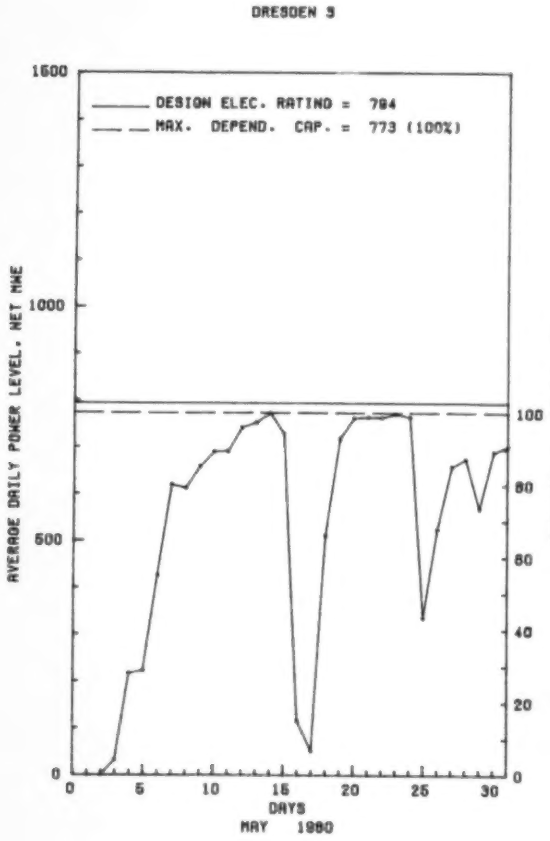
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-13/03L-0	03/28/80	04/22/80	During refueling outage of Unit 3, a leakage of 967.86 SCFH occurred on "B" outboard check valve.
80-16/03L-0	04/02/80	04/28/80	During leak rate test of HPCI suction line, leakage was 1,370.7 SCFH, which is in excess of TS for single containment isolation valve.
80-17/03L-0	04/04/80	04/28/80	During refueling outage, during oper. survey test, the diesel generator failed to start.
80-19/03L-0	04/20/80	05/14/80	During refueling outage performing target rock relief vlv press switch cal. surv. PS-3-203-3A tripped at 1098 psig increasing.
80-20/03L-0	04/23/80	05/14/80	During refueling outage, 3B Core Spray relief valve found leaking.
80-21/01T-0	04/25/80	05/08/80	Combinations of 3A Target Rock Valve and 3B, 3C and 3E Electromatic relief valves failed to open at rated reactor pressure.

REVIEWED BY OMPA LOEB: Earl [illegible] DATE: 7/5/80

REVIEWED BY NRR DOR: RBBevan DATE: 7/8/80

DRESDEN 3

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-249
2. Utility Contact: S. A. Cieszkiewicz (815) 942-2920 x263
3. Licensed Thermal Power (MWt): 2527
4. Nameplate Rating (Gross MWe): 920 x 0.9 = 828
5. Design Electrical Rating (Net MWe): 794
6. Maximum Dependable Capacity (Gross MWe): 812
7. Maximum Dependable Capacity (Net MWe): 773
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	3,647.0	77,680.0
12. Hours Reactor Was Critical	691.2	1,476.2	57,241.6
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	631.9	1,410.6	54,735.2
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,306,266	2,633,992	104,905,660
17. Gross Electrical Energy (MWH)	415,519	840,294	34,269,878
18. Net Electrical Energy (MWH)	394,682	784,617	32,477,908
19. Unit Service Factor	84.9	38.7	70.5
20. Unit Availability Factor	84.9	38.7	70.5
21. Unit Capacity Factor (MDC Net)	68.6	27.8	54.1
22. Unit Capacity Factor (DER Net)	66.8	27.1	52.7
23. Unit Forced Outage Rate	8.3	5.3	16.2

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
2	02/03/80	S	55.0	C	4				REFUELING OUTAGE.
3	05/03/80	F	6.6	H	9				TURBINE TRIP - HI #6 BEARING VIBRATION (RX REMAINED CRITICAL).
4	05/03/80	F	1.3	H	9				TURBINE TRIP - 3B MOISTURE SEPARATOR HI HI (RX REMAINED CRITICAL).
5	05/05/80	F	9.5	G	3				RX SCRAM - RX LOW LEVEL (INSTRUMENT RACK JARRED).
6	05/15/80	F	9.6	A	9				ECH OIL LEAK (RX REMAINED CRITICAL).
7	05/16/80	F	5.2	A	9				EHC OIL LEAK (RX REMAINED CRITICAL).
8	05/17/80	F	15.6	A	9				#2 TURBINE CONTROL VALVE STUCK OPEN (RX REMAINED CRITICAL).
9	05/23/80	F	9.3	H	3				TURBINE TRIPPED ON MOISTURE SEPARATOR "C" HI HI LEVEL - RX SCRAMMED ON TURBINE STOP VALVE CLOSURE.

SUMMARY

ON 05/03/80, THE UNIT WAS PUT ON LINE AFTER A REFUELING OUTAGE. IT RAN NORMALLY UNTIL 05/15/80 WHEN IT WAS SHUTDOWN TO REPAIR AN EHC OIL LEAK'. IT WAS ALSO TAKEN OFF LINE TO REPAIR A STUCK OPEN CONTROL VALVE ON 05/17/80. THE UNIT OPERATED NORMALLY FOR THE REST OF THE MONTH.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

NRC POWER RESTRICTIONS: NONE

DUANE ARNOLD

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Iowa/Linn 8 miles NW of Cedar Rapids
2.	Electric Reliability Council	Mid-Continent Area Reliability Coordination Agreement
3.	Type of Reactor	BWR
4.	Power Level (MWT)	1658
5.	Design Electrical Rating (MWE Net)	538
6.	Date of Initial Criticality	March 23, 1974
7.	Date of Commercial Operation	February 1, 1975
8.	Condenser Cooling Method	Cooling Towers
9.	Condenser Cooling Water Source	Cedar Rapids River Circulating Water System

Utility & Contractor Information

10.	Licensee	Iowa Electric Light and Power
11.	Corporate Address	IE Towers, P. O. Box 351 Cedar Rapids, Iowa 52406
12.	Corporate Contact	D. Arnold, President
13.	Architect/Engineer	Bechtal
14.	Nuclear Steam Supply System	GE
15.	Constructor	Bechtal

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	W. S. LITTLE
19.	IE Resident Inspector	W. F. CHRISTIANSON
20.	Licensing Project Manager	T. Kevern
21.	Docket Number	50-331
22.	License Number/Date of Issue	DPR-49 issued 2/22/74
23.	Public Document Room	Reference Service Cedar Rapids Public Library 428 Third Avenue, S.E. Cedar Rapids, Iowa 52401

INSPECTION STATUS

ENFORCEMENT STATUS

TECHNICAL SPECIFICATION SECTION 6.8.2 STATES IN PART: "PROCEDURES...AND CHANGES THERETO, SHALL BE REVIEWED...AND APPROVED...PRIOR TO IMPLEMENTATION EXCEPT AS NOTED IN SECTION 6.8.3..." TECHNICAL SPECIFICATION SECTION 6.8.3 STATES IN PART: "TEMPORARY MINOR CHANGES TO PROCEDURES...WHICH DO NOT CHANGE THE INTENT...MAY BE MADE WITH THE CONCURRENCE OF TWO MEMBERS OF THE PLANT MANAGEMENT..." CONTRARY TO THE ABOVE, BETWEEN MARCH 10 AND MARCH 18, 1980, NUMEROUS CHANGES TO PROCEDURE RP83/IE-2 REVISION 1 WERE MADE WITHOUT EITHER PRIOR APPROVAL OR CONCURRENCE BY TWO MEMBERS OF THE PLANT MANAGEMENT.
(8004 INFRACTION)

TECHNICAL SPECIFICATION 3.3.1.B.2 REQUIRES, IN PART, THAT PRIOR TO RELEASE OF EACH BATCH OF LIQUID EFFLUENT, A SAMPLE SHALL BE TAKEN FROM THAT BATCH TO DETERMINE THAT CONCENTRATIONS OF GROSS RADIOACTIVITY ARE IN COMPLIANCE WITH TECHNICAL SPECIFICATION 2.3.1.B PRIOR TO RELEASE. CONTRARY TO THIS REQUIREMENT, THE RESULTS OF THE RADIOACTIVE CONCENTRATIONS OF THE EFFLUENT RELEASED TO AN UNRESTRICTED AREA WAS NOT KNOWN UNTIL AFTER THE RELEASE OCCURRED.
(8005 INFRACTION)

10 CFR 50.72 REQUIRES, IN PART, THAT EACH LICENSEE OF A NUCLEAR POWER REACTOR LICENSED UNDER 10 CFR 50.21 OR 50.22 SHALL NOTIFY THE NRC OPERATIONS CENTER AS SOON AS POSSIBLE AND IN ALL CASES WITHIN ONE HOUR BY TELEPHONE OF ANY UNPLANNED RADIOACTIVE RELEASE. CONTRARY TO THIS REQUIREMENT, NO NOTIFICATION OF AN UNPLANNED RELEASE OF APPROXIMATELY EIGHT MICROCURIES OF RADIOACTIVITY RELEASED TO AN UNRESTRICTED AREA ON MARCH 12 AND 13, 1980, WAS GIVEN TO THE NRC OPERATIONS CENTER.
(8005 INFRACTION)

TECHNICAL SPECIFICATION SECTION 6.4.2 STATES IN PART, "...FIRE BRIGADE TRAINING SESSIONS...SHALL BE HELD AT LEAST QUARTERLY." CONTRARY TO THE ABOVE, FIRE BRIGADE TRAINING HAD NOT BEEN HELD DURING THE PERIOD OF JULY 1978 TO JUNE 1979.
(8003 INFRACTION)

TECHNICAL SPECIFICATION SECTION 6.8 STATES IN PART, "DETAILED WRITTEN PROCEDURES INVOLVING NUCLEAR SAFETY...SHALL BE PREPARED...ALL PROCEDURES SHALL BE ADHERED TO." INCLUDED IN SECTION 6.8.1 IS "FIRE PROTECTION PLAN IMPLEMENTATION." CONTRARY TO THE ABOVE THE MONTHLY FIRE EXTINGUISHER INSPECTIONS REQUIRED BY THE DAEC FIRE PLAN AND INSPECTION PROCEDURE IP-013/IE-3 HAVE NOT BEEN PERFORMED FROM 1977 TO DATE.
(8003 INFRACTION)

TECHNICAL SPECIFICATION SECTION 6.8 STATES IN PART, "DETAILED WRITTEN PROCEDURES INVOLVING NUCLEAR SAFETY, INCLUDING APPLICABLE CHECK-OFF LISTS...SHALL BE PREPARED..." OPERATING INSTRUCTION NO. 53 "STANDBY LIQUID CONTROL SYSTEM" APPENDIX I INDICATED THAT AIR SPARGER, AIR INLET VALVE U-26-11 IS TO BE LOCKED CLOSED. CONTRARY TO THE ABOVE, STP 44C001 "STANDBY LIQUID CONTROL SYSTEM BORON CONCENTRATION" DOES NOT CONTAIN SUFFICIENT DIRECTION AND/OR CHECK SHEETS TO PREVENT VALVE U-26-11 FROM BEING LEFT UNLOCKED AT THE CONCLUSION OF THE PROCEDURE. THIS WAS THE CASE WHEN THE INSPECTOR OBSERVED VALVE U-26-11 TO BE UNLOCKED AFTER THE COMPLETION OF STP 44C001.
(8003 DEFICIENCY)

10 CFR 50, APPENDIX B, CRITERIA XVI, "CORRECTIVE ACTION" STATES IN PART..."MEASURES SHALL BE ESTABLISHED TO ASSURE THAT CONDITIONS ADVERSE TO QUALITY...ARE PROMPTLY IDENTIFIED AND CORRECTED..." IOWA ELECTRIC QUALITY DIRECTIVE 1316.1 "CORRECTIVE ACTION" STATES: "SYSTEMS SHALL BE IMPLEMENTED TO ASSURE THAT CONDITIONS ADVERSE TO QUALITY ARE PROMPTLY IDENTIFIED AND CORRECTED AS SOON AS POSSIBLE." CONTRARY TO THE ABOVE, BETWEEN MARCH 10 AND MARCH 18, 1980 NUMEROUS ITEMS WERE IDENTIFIED, DURING QC INSPECTIONS DEALING WITH FAILURE TO FOLLOW PROCEDURE RP83/IE-2 REVISION 1, WITH NO PROMPT CORRECTIVE ACTION BEING TAKEN TO CORRECT THE SITUATION.
(8004 INFRACTION)

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE			

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

Unit operating routinely after repair of Recirculation Motor-Generator Set Motor. The unit had operated for approximately one week on a single recirc. loop.

LAST IE SITE INSPECTION DATE: March 24-26; April 10-12 and 22-25, 1980

INSPECTION REPORT NO: 80-07

SUMMARY

INSPECTION SUMMARY

Inspection on March 18-21, (80-04): Routine, unannounced inspection of refueling maintenance activities. Also follow-up on unplanned release of radioactive material from the site. The inspection involved 29 inspector hours onsite by two NRC inspectors and 4 inspector hours at the corporate office by one NRC inspector. Of the areas inspected, two items of noncompliance (failure to review and approve temporary procedure changes and failure to take prompt corrective action on items identified during QC inspections) were identified.

Inspection on April 7-11, (80-06): Review of specifications and procedures, observation of work, and review of documentation for Internal Torus Modifications; review of ultrasonic examination records of jet pump beams. The inspection involved 32 inspector hours onsite by one NRC inspector. No items of noncompliance or deviations were identified.

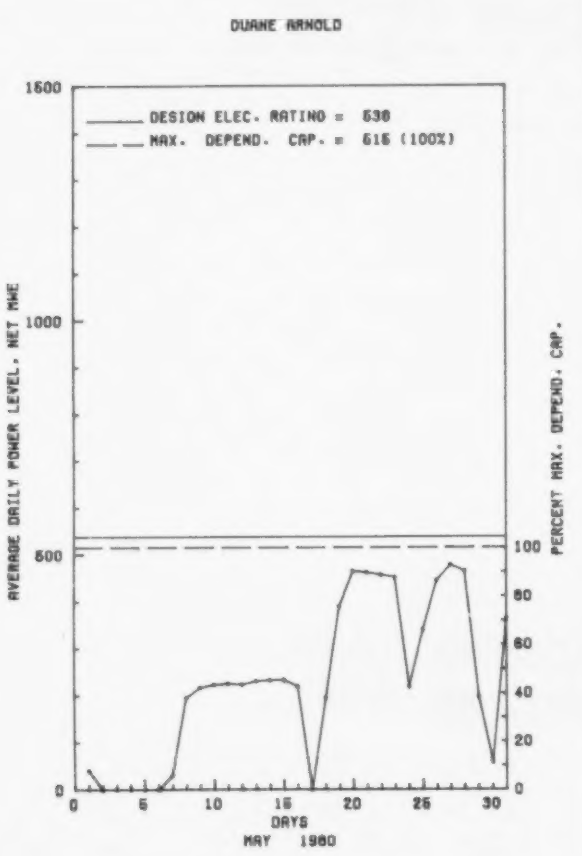
Inspection on March 24-26; April 10-12 and 22-25, (80-07): The inspection was broken into two segments. First was a special unannounced inspection and subsequent followup on Small Break Loss of Coolant Accident (SBLOCA) and High Energy Line Break Outside Containment (HELB) procedures and training; modifications to containment isolation logic required by IE Bulletins 79-05, 79-06, and 79-08; and followup on progress in regard to IE Bulletin 79-01B (March 24-26 and April 10-12, 1980). The second segment was a routine, unannounced inspection of facility operations, IE Circular followup and OII followup. The inspection involved 140 inspector hours onsite by 5 NRC inspectors and 12 inspector hours offsite by 3 NRC inspectors. In the areas inspected, no items of noncompliance were identified.

REVIEWED BY OMPA LOEB: Earl Hill **DATE:** 7/3/80

REVIEWED BY NRR DOR: T. Kevern **DATE:** 7/5/80

DUANE ARNOLD

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-331
2. Utility Contact: J. Van Sickel (319) 851-5611
3. Licensed Thermal Power (MWt): 1658
4. Nameplate Rating (Gross MWe): 663 x 0.9 = 597
5. Design Electrical Rating (Net MWe): 538
6. Maximum Dependable Capacity (Gross MWe): 545
7. Maximum Dependable Capacity (Net MWe): 515
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): 50% (05/07/80 TO 05/16/80)
10. Reasons for Restrictions, If Any:
TECH SPEC RESTRICT FOR OPER WITH ONE RECIRC SYS INOP.

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	3,647.0	46,727.3
12. Hours Reactor Was Critical	590.5	1,848.7	32,412.8
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	555.0	1,766.4	31,585.5
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	563,184	2,041,176	39,170,736
17. Gross Electrical Energy (MWH)	183,696	696,326	13,120,692
18. Net Electrical Energy (MWH)	170,283	649,673	12,264,737
19. Unit Service Factor	74.6	48.4	67.6
20. Unit Availability Factor	74.6	48.4	67.6
21. Unit Capacity Factor (MDC Net)	44.4	34.6	51.9
22. Unit Capacity Factor (DER Net)	42.5	33.1	48.8
23. Unit Forced Outage Rate	23.2	8.7	20.3

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
5	05/01/80	F	151.3	A	1	80-01[illegible]	CB	MOTORX	SHUTDOWN DUE TO THE FAILURE OF THE "A" RECIRCULATION SYSTEM M-G SET DRIVE MOTOR.
6	05/17/80	S	21.3	H	1				SHUTDOWN TO PLACE "A" RECIRC. SYSTEM BACK IN SERVICE.
7	05/24/80	S	.0	B	5				POWER REDUCED TO PERFORM MAINTENANCE ON THE "A" FEEDWATER PUMP.
8	05/29/80	F	16.4	A	1				POWER REDUCED AND PLANT SHUTDOWN DUE TO HIGH/LOW LEVEL ALARM ON "B" RECIRC. PUMP MOTOR.

SUMMARY

THE PLANT SHUTDOWN ON 05/01/80 FOR RECIRC MOTOR REPAIR. THE UNIT RETURNED TO SERVICE ON 05/07/80 AND RAN AT 50%, BECAUSE ONLY 1 RECIRC SYSTEM WAS AVAILABLE, UNTIL SHUTDOWN ON 05/17/80 FOR MORE RECIRC SYSTEM REPAIR. THE PLANT THEN RAN AT NEAR FULL POWER EXCEPT FOR REDUCTIONS ON 05/24/80 FOR FDWR PUMP REPAIR AND ON 05/29/80 FOR MORE RECIRC SYSTEM REPAIR.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

FARLEY 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Alabama/Houston 24 miles SE of Dothan
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2652
5.	Design Electrical Rating (MWE Net)	829
6.	Date of Initial Criticality	August 9, 1977
7.	Date of Commercial Operation	December 1, 1977
8.	Condenser Cooling Method	Once Through With Forced Draft Cooling Towers
9.	Condenser Cooling Water Source	Chattahoochee River

Utility & Contractor Information

10.	Licensee	Alabama Power Company
11.	Corporate Address	600 North 18th Street Birmingham, Alabama
12.	Corporate Contact	F. L. Clayton, Jr. Senior Vice President (205-323-5341)
13.	Architect/Engineer	Bechtel/SSI
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Daniel

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. Lewis
16.	IE Section Chief	R. D. Martin
• 19.	IE Resident Inspector	W. H. Bradford (205-899-3386)
20.	Licensing Project Manager	E. A. Reeves
21.	Docket Number	50-348
22.	License Number/Date of Issue	NPF-2, 6/25/77
23.	Public Document Room	G. S. Houston Memorial Library 212 W. Berdeshaw Street Dothan, Alabama 36301

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

100% POWER OPERATION.

LAST IE SITE INSPECTION DATE: APRIL 28 - MAY 30, 1980

INSPECTION REPORT NO: 50-348/80-13

SUMMARY

INSPECTION SUMMARY

INSPECTION APRIL 21-25 (80-09): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 70 INSPECTOR-HOURS ON SITE IN THE AREAS OF RADIOACTIVE WASTE MANAGEMENT FOR UNIT 2 INCLUDING THE INSTALLATION OF THE LIQUID AND GASEOUS WASTE SYSTEMS, REVIEW OF PREOPERATIONAL TEST PROCEDURES, INSTALLATION OF PROCESS AND EFFLUENT MONITORS, CHEMISTRY AND HEALTH PHYSICS PROCEDURES AND HEALTH PHYSICS AND SECURITY TRAINING FOR NEW EMPLOYEES AND CONTRACTORS. OF THE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION MARCH 31 - APRIL 25 (80-10): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 155 INSPECTOR-HOURS ON SITE IN THE AREAS OF HOT FUNCTIONAL TESTING, PRE-OPERATIONAL TESTING, UNIT 2 PLANT TESTING STATUS, UNIT 2 PLANT TOUR, UNIT 1 PLANT OPERATION, REVIEW OF LICENSEE EVENT REPORTS, IEB FOLLOWUP, PLANT SECURITY, OPEN ITEMS, AND REVIEW OF ADMINISTRATIVE CONTROLS. OF THE TEN AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

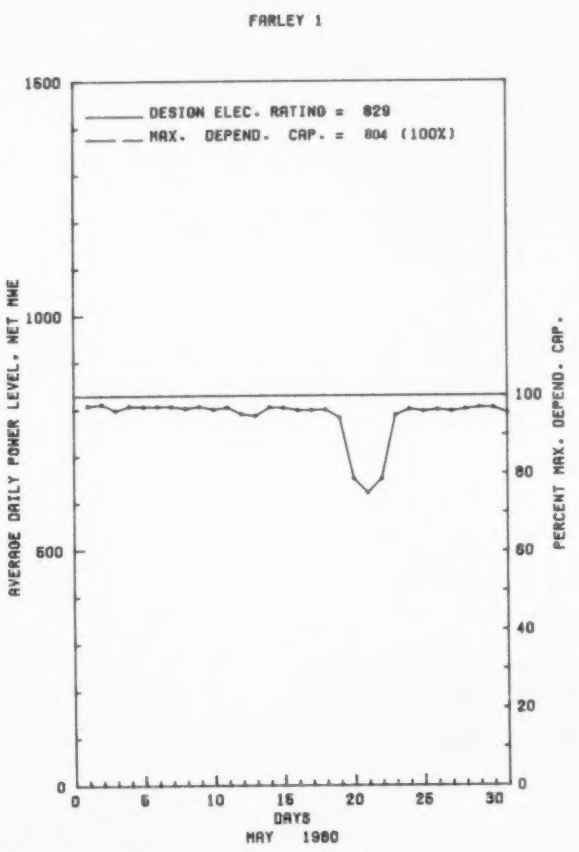
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
80-021/ 03L-0	03/16/80	04/09/80		RCS LEAKAGE TEST NOT DONE WITHIN 90 HRS. (116HR) OF LAST PERFORMANCE, DUE TO PERSONNEL ERROR. TEST DONE SATISFACTORILY ONCE IDENTIFIED.
80-023/ 03L-0	03/29/80	04/28/80		BIT TANK INOPERABLE WHEN BORIC ACID CRYSTALS CLOGGED SUCTION LINE RECIRCULATION PUMP 1A LOSS OF BIT SURGE TANK LEVEL COMPOUNDED PROBLEM LINE CLEARED AND SYSTEM RETURNED TO SERVICE.
80-024/ 03L-0	03/30/80	04/22/80		WHILE CONTROLLING S/G "B" LEVEL IN MANUAL, TRIPPED FEEDPUMP ON HIGH LEVEL. ROD INSERTION AND RESUMING FEED TO S/G CAUSED TAVE TO DECREASE TO 530 DEGREES F.
80-025/ 03L-0	03/31/80	04/29/80		ON THREE ATTEMPTS, DIESEL GENERATOR 1C FAILED TO ACCELERATE TO RATED SPEED IN UNDER 10 SECONDS. ONLY ANOMIALLY WAS LUBE OIL SUMP TEMPERATURE AT 110 VS 120 RAISED TEMPERATURE.
80-026/ 03L-0	04/05/80	05/05/80		ONE OVERTEMPERATURE DELTA-T PROTECTION CHANNEL INOPERABLE.
80-027/ 03L-0	04/08/80	05/05/80		ONE TURBINE FIRST STAGE PRESSURE PROTECTION CHANNEL INOPERABLE.

REVIEWED BY OMPA LOEB: T. C. Cintula **DATE:** 7·3·80

REVIEWED BY NRR DOR: Ed Reeves **DATE:** 7-7-80

FARLEY 1

AVERAGE POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-348
2. Utility Contact: W. G. Hairston,III (205) 899-5156
3. Licensed Thermal Power (MWt): 2652
4. Nameplate Rating (Gross MWe): 1045 x 0.85 = 888
5. Design Electrical Rating (Net MWe): 829
6. Maximum Dependable Capacity (Gross MWe): 845
7. Maximum Dependable Capacity (Net MWe): 804
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
MDC GROSS & NET DETERMINED FROM OPERATING EXPERIENCE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	3,647.0	21,911.0
12. Hours Reactor Was Critical	744.3	3,210.6	14,146.1
13. Reactor Reserve Shutdown Hours	0	436.4	1,841.7
14. Hours Generator On-Line	744.0	3,147.7	13,738.2
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,927,148	7,958,932	34,556,947
17. Gross Electrical Energy (MWH)	614,696	2,562,364	11,086,402
18. Net Electrical Energy (MWH)	583,600	2,431,702	10,491,118
19. Unit Service Factor	100.0	86.3	62.7
20. Unit Availability Factor	100.0	86.3	62.7
21. Unit Capacity Factor (MDC Net)	97.6	82.9	59.6
22. Unit Capacity Factor (DER Net)	94.6	80.4	57.8
23. Unit Forced Outage Rate	.0	3.9	6.8

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
REFUELING OUTAGE, 9/80, APPROX 8 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
039	05/03/80	S	.0	B	5		HA	VALVEX	UNIT WAS REDUCED TO AND HELD AT 775 MWE FOR MAIN TURBINE GOVERNOR VALVE TESTING.
040	05/10/80	S	.0	B	5		HA	VALVEX	UNIT WAS REDUCED TO AND HELD AT 770 MWE FOR MAIN TURBINE GOVERNOR VALVE TESTING.
041	05/11/80	F	.0	A	5	80-031/03L-0	ID	INSTRU	UNIT WAS RAMPING DOWN TO BE TAKEN OFF LINE IN COMPLIANCE WITH THE TECHNICAL SPECIFICATION FOR LOSS OF DATA CABINETS A AND B DIRECT ROD POSITION INDICATION. AT 782 MWE, ONE CHANNEL OF THE DRPI WAS RESTORED AND RAMP-DOWN TERMINATED.
042	05/12/80	S	.0	B	5		HH	XXXXXX	UNIT WAS REDUCED TO AND HELD AT 770 MWE TO REPAIR A LUBE OIL GASKET LEAK ON 1C CONDENSATE PUMP.
043	05/19/80	S	.0	B	5		HH	PUMPXX	UNIT WAS REDUCED TO AND HELD AT 700 MWE TO REPLACE 1A CONDENSATE PUMP AND MOTOR DUE TO EXCESSIVE VIBRATION.
044	05/21/80	S	.0	B	5		CH	PIPEXX	UNIT WAS REDUCED TO AND HELD AT 550 MWE TO REPAIR A PINHOLE LEAK IN A SOCKET WELD IN THE LOW PRESSURE STEAM SUPPLY LINE TO 1A SGFP BETWEEN THE LOW PRESSURE STOP VALVE AND SEAT DRAIN TO THE CONDENSER.
045	05/21/80	S	.0	B	5		HH	PUMPXX	UNIT WAS RAISED TO & HELD AT 700 MWE TO COMPLETE REPLACEMENT OF 1A CONDENSATE PUMP AND MOTOR DUE TO BEARING FAILURES.
046	05/31/80	S	.0	B	5		HA	VALVEX	UNIT WAS REDUCED TO AND HELD AT 760 MWE FOR MAIN TURBINE GOVERNOR VALVE TESTING.

SUMMARY

THE UNIT OPERATED AT FULL POWER EXCEPT FOR 8 REDUCTIONS FOR TURBINE GOV. VALVE TESTING AND FOR MAINTENANCE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

FITZPATRICK

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	New York/Oswego 8 miles NE of Oswego
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2436
5.	Design Electrical Rating (MWE Net)	821
6.	Date of Initial Criticality	November 17, 1974
7.	Date of Commercial Operation	July 28, 1975
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Lake Ontario

Utility & Contractor Information

10.	Licensee	Power Authority of the State of New York
11.	Corporate Address	10 Columbus Circle New York, New York 10019
12.	Corporate Contact	P. Lyon, Manager of Nuclear Operations
13.	Architect/Engineer	Stone & Webster
14.	Nuclear Steam Supply System	GE
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner (X246)
18.	IE Section Chief	H. B. KISTER
19.	IE Resident Inspector	W. H. BAUNACK
20.	Licensing Project Manager	P. Polk (X7872) 5)
21.	Docket Number	50-333
22.	License Number/Date of Issue	DPR-59, 10/17/74
23.	Public Document Room	State University College of Oswego Penfield Library - Documents Oswego, NY 13126 (315)341-2323

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO THE JAMES A. FITZPATRICK SECURITY PLAN, A BADGED NON-REGULAR EMPLOYEE WAS NOT PROPERLY SEARCHED. (8002 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

FACILITY ITEMS (PLANS AND PROCEDURES):

Mark I torus modifications are in progress.

MANAGERIAL ITEMS:

Mr. P. Bayne assumed the position of Sr. Vice President Nuclear Generation on May 29, 1980. The position was vacated on May 28, 1980 by Mr. P. Lyon.

PLANT STATUS:

The facility is currently in a 65 day refueling outage which began on May 12, 1980.

LAST IE SITE INSPECTION DATE: 5/23/80

INSPECTION REPORT NO: 50-333/80-09

SUMMARY

INSPECTION SUMMARY

50-333/80-04 - Mar 31-Apr 4: Routine, unannounced inspection of plant operations as part of a turnover of NRC Principle/Resident Inspectors, licensee progress on pipe hanger installations and modifications and licensee action on previous inspection items. This inspection also included an independent review of the instrumentation required to be environmentally qualified in accordance with IE Bulletin 79-01B. The inspection involved 91.5 hours onsite by two region-based inspectors and one resident inspector trainee and 10 hours onsite by one Reactor Projects Section Chief. No items of noncompliance were identified.

50-333/80-05 - Apr 7-11: Routine, unannounced inspection by a regionally-based inspector (37 hours) of licensee action on previous inspection findings; administrative control of safety related calibrations; surveillance calibrations of safety related components and equipment required by Technical Specifications; calibrations required by Technical Specifications of components and equipment associated with safety related systems and/or functions; calibration and control of test equipment; witnessing of calibrations; technician qualifications; and, facility tour. Three items of noncompliance were identified: (Failure to perform periodic review of procedures; Failure to follow administrative procedures; and Failure to adequately implement calibration program for safety related instruments).

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

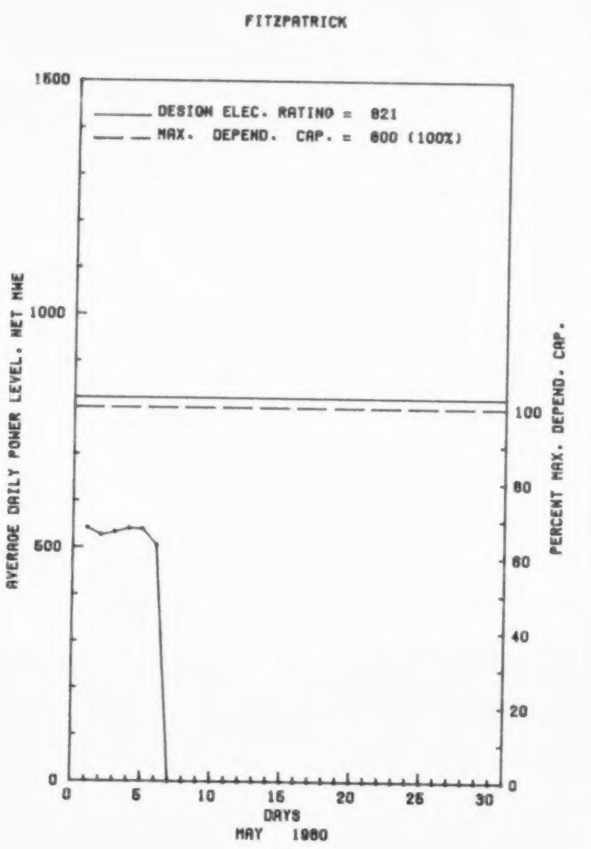
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-028/03L	03/24/80	04/23/80	1 loop of torus cooling declared inoperable hanger modification made to correct condition
80-029/03L	03/25/80	04/24/80	Condensate storage tank level switch drift
80-030/03L	03/27/80	04/25/80	Fire seals missing or requiring resealing identified during performance of required surveillance
80-031/03L	04/05/80	05/01/80	Drywell to torus differential pressure out of limits for 35 minutes
80-032/04L	04/08/80	05/08/80	Release of halogens & particulates with half lives greater than 8 days was 4.92% of ETS because of packing leak in RHR system
80-033/03L	04/09/80	05/09/80	Diesel fire pump inop for filb connection replacement
80-034/03L	04/10/80	05/09/80	HPCI isolation valve failed to close during surveillance testing

REVIEWED BY OMPA LOEB: Earl C. [illegible] **DATE:** 7/1/80

REVIEWED BY NRR DOR: Philip J. Polk **DATE:** 7/7/80

FITZPATRICK

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

ITEM 21 CALCULATED BY USING A WEIGHTED AVERAGE

1. Docket Number: 50-333
2. Utility Contact: E. Zufelt (315) 342-3840
3. Licensed Thermal Power (MWt): 2436
4. Nameplate Rating (Gross MWe): 981 x 0.9 = 883
5. Design Electrical Rating (Net MWe): 821
6. Maximum Dependable Capacity (Gross MWe): 830
7. Maximum Dependable Capacity (Net MWe): 800
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	3,647.0	42,480.0
12. Hours Reactor Was Critical	142.6	2,936.1	29,566.3
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	142.6	2,893.0	28,631.0
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	230,640	5,862,024	57,050,218
17. Gross Electrical Energy (MWH)	79,450	2,019,020	19,372,720
18. Net Electrical Energy (MWH)	76,545	1,953,370	18,745,945
19. Unit Service Factor	19.2	79.3	67.4
20. Unit Availability Factor	19.2	79.3	67.4
21. Unit Capacity Factor (MDC Net)	12.9	67.0	60.4
22. Unit Capacity Factor (DER Net)	12.5	65.2	53.7
23. Unit Forced Outage Rate	8.6	5.4	19.7

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 07/15/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
13	05/06/80	F	13.4	A	3				REACTOR SCRAM WHILE TESTING CIV/ISV VALVES. MALFUNCTION IN THE EHC SYSTEM.
14	05/07/80	S	588.0	C	4				STARTED REFUELING OUTAGE AT 1200 HOURS.

SUMMARY

POWER WAS BEING INCREASED AT THE PRECONDITIONING RATE AT THE BEGINNING OF THIS REPORT PERIOD. ON MAY 6, 1980, THE REACTOR SCRAMMED DUE TO A MALFUNCTION IN THE EHC SYSTEM. STARTED SCHEDULED REFUELING OUTAGE AT 1200 HOURS ON MAY 7, 1980.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

FORT CALHOUN

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Nebraska/Washington 19 miles N of Omaha
2.	Electric Reliability Council	Mid-Continent Area Reliability Coordination Agreement
3.	Type of Reactor	PWR
4.	Power Level (MWT)	1420
5.	Design Electrical Rating (MWE Net)	457
6.	Date of Initial Criticality	August 6, 1973
7.	Date of Commercial Operation	June 20, 1974
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Missouri River

Utility & Contractor Information

10.	Licensee	W. C. Jones Division Manager – Production Operations Omaha Public Power District
11.	Corporate Address	1623 Harney Street Omaha, Nebraska 68102
12.	Corporate Contact	T. E. Short Assistant General Manager
13.	Architect/Engineer	Gibbs & Hill, Inc.
14.	Nuclear Steam Supply System	Combustion Engineering
15.	Constructor	Gibbs & Hill, Durham & Richardson, Inc.

Regulatory Information

16.	IE Region responsible	IV
17.	IE Branch Chief	G. L. Madsen
18.	IE Section Chief	T. F. Westerman
19.	IE Resident Inspector	D. Kelley
20.	Licensing Project Manager	P. C. Wagner
21.	Docket Number	50-285
22.	License Number/Date of Issue	DPR-40, 5/24/73
23.	Public Document Room	W. Dale Clark Library 215 S. 15th Street Omaha, Nebraska 68102

INSPECTION STATUS

ENFORCEMENT STATUS

SECTION 5.8.1 OF YOUR TECHNICAL SPECIFICATION REQUIRES THAT WRITTEN PROCEDURES BE ESTABLISHED AND IMPLEMENTED IN ACCORDANCE WITH NRC REG. GUIDE 1-33, APPENDIX A, SECTION 7(E)(2), "RADIATION SURVEYS." OPPD STANDING ORDER NO. T-8 REQUIRES THAT ROUTINE SURVEYS BE CONDUCTED EACH MONTH IN ROOM 15A. CONTRARY TO THE ABOVE, NO SURVEY WAS CONDUCTED DURING THE MONTH OF APRIL 1980.
(8010 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

FACILITY ITEMS (PLANS AND PROCEDURES):

MANAGERIAL ITEMS:

(1) Safety Audit and Review Committee review of Facility License Changes. (76-16)

PLANT STATUS:

No significant information

LAST IE SITE INSPECTION DATE: May 5-8, 12-15, and 22-23, 1980

INSPECTION REPORT NO: 50-285/80-09

SUMMARY

INSPECTION SUMMARY

Inspection on May 5-8, 12-15, and 22-23, 1980 (80-09): Routine, unannounced inspection of the licensee's (1) Progress on IEB 79-14; (2) Preparation for startup after refueling; (3) Follow-up on an allegation; (4) Follow-up on items of noncompliance and deviations; (5) Follow-up on unresolved items; and (6) Reactor coolant pump cover stud deterioration. Of the six areas inspected no items of noncompliance or deviations were identified.

REPORTS RECEIVED FROM LICENSEE

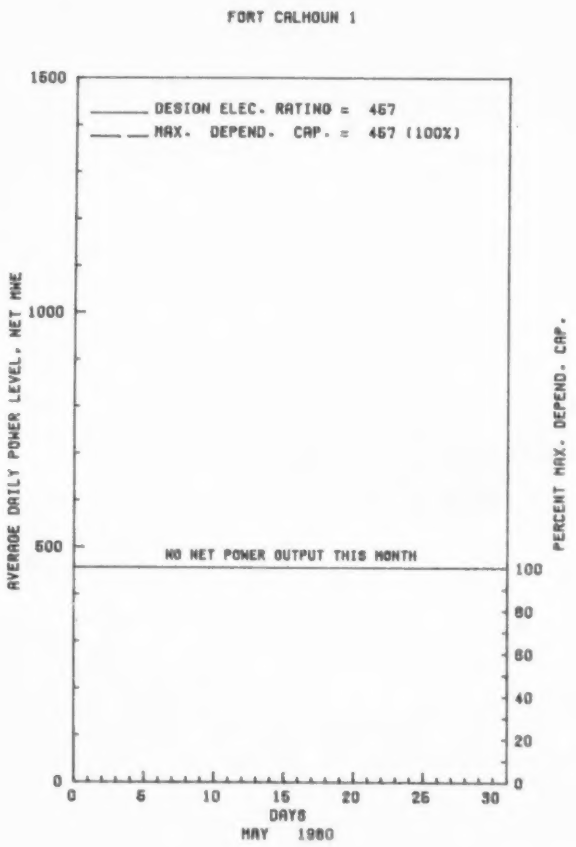
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE			

REVIEWED BY OMPA LOEB: L. T **DATE:** 7/3/80

REVIEWED BY NRR DOR: PC Wagner **DATE:** 7/7/80

FORT CALHOUN

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-285
2. Utility Contact: B. J. Hickle (402) 536-4413
3. Licensed Thermal Power (MWt): 1420
4. Nameplate Rating (Gross MWe): 591 x 0.85 = 502
5. Design Electrical Rating (Net MWe): 457
6. Maximum Dependable Capacity (Gross MWe): 481
7. Maximum Dependable Capacity (Net MWe): 457

ITEM 21 CALCULATED BY USING A WEIGHTED AVERAGE

8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	3,647.0	58,584.0
12. Hours Reactor Was Critical	.0	432.9	44,834.5
13. Reactor Reserve Shutdown Hours	.0	.0	1,309.5
14. Hours Generator On-Line	.0	430.5	43,881.6
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	574,731	53,889,758
17. Gross Electrical Energy (MWH)	0	190,604	17,856,834
18. Net Electrical Energy (MWH)	0	180,935	16,868,452
19. Unit Service Factor	.0	11.8	74.9
20. Unit Availability Factor	.0	11.8	74.9
21. Unit Capacity Factor (MDC Net)	.0	10.9	63.4
22. Unit Capacity Factor (DER Net)	.0	10.9	63.0
23. Unit Forced Outage Rate	.0	.0	4.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 06/07/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-01	01/18/80	S	744.0	C	4		RC	FUELXX	REACTOR SHUTDOWN FOR ANNUAL REFUELING AND MAINTENANCE OUTAGE.

SUMMARY

THE REACTOR WAS SHUTDOWN FOR ANNUAL REFUELING AND MAINTENANCE OUTAGE.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

best copy available

FORT ST. VRAIN

FACILITY DATA

Facility Description

1. Location (state/countv) (nearest population center)	Colorado/Weld 35 miles N of Denver (near Platteville)
2. Electric Reliability Council	Western Systems Coordinating Council
3. Type of Reactor	HTGR
4. Power Level (MWT)	842
5. Design Electrical Rating (MWE Net)	330
6. Date of Initial Criticality	January 31, 1974
7. Date of Commercial Operation	July 1, 1979
8. Condenser Cooling Method	Towers
9. Condenser Cooling Water Source	South Platte River/Shallow Wells

Utility & Contractor Information

10. Licensee	Public Service Company of Colorado
11. Corporate Address	P. O. Box 840 Denver, Colorado 80201
12. Corporate Contact	C. K. Millen Senior Vice President
13. Architect/Engineer	Sergent & Lundy
14. Nuclear Steam Supply System	General Atomic Company
15. Constructor	General Atomic Company

Regulatory Information

16. IE Region responsible	IV
17. IE Branch Chief	G. L. Madsen
18. IE Section Chief	T. F. Westerman
19. IE Resident Inspector	M. W. Dickerson (303) 785-2281
20. Licensing Project Manager	G. Kuzmycz
21. Docket Number	50-267
22. License Number/Date of Issue	DPR-34, 12/21/73
23. Public Document Room	Greeley Public Library City Complex Building Greeley, Colorado 80631

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO TS 4.6.1 LCO, BOTH DIESEL GENERATOR DAY TANKS CONTAINED LESS THAN THE REQUIRED 500 GALLONS WHILE THE REACTOR WAS OPERATED AT POWER.
(8008 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

Areas related to potential equipment modification, which are under review and discussion with the NRC include: helium circulator auxiliaries; accident analysis relating to outlet temperature dispersion and orifice settings; hangers for 2" and smaller piping; and plant oscillations.

FACILITY ITEMS (PLANS AND PROCEDURES):

IMPLEMENTATION OF 10 CFR 73.55 PROVISIONS WERE IN EFFECT 3/21/80.

MANAGERIAL ITEMS:

PLANT STATUS:

Plant is operating at 50% power.

LAST IE SITE INSPECTION DATE: April 1-30, 1980

INSPECTION REPORT NO: 50-267/80-08

SUMMARY

INSPECTION SUMMARY

Inspection April 15-18, 1980 (80-09): Routine, unannounced inspection of procedures; surveillance of pipe support and restraint systems; maintenance program; and plant tour. No items of noncompliance or deviations were identified.

Inspection April 1-30, 1980 (80-08): Routine, announced inspection of Review of Previous Items of Noncompliance; Surveillance; Maintenance; Operational Safety Verification; and Review of Bulletins/Circulars. Within the five (5) areas inspected, one item of noncompliance was identified (infraction - failure to meet Technical Specification for Diesel Generator Day Tank Levels)

NRC POWER RESTRICTIONS: Restricted to 70% of Licensed Thermal Power.

REPORTS RECEIVED FROM LICENSEE

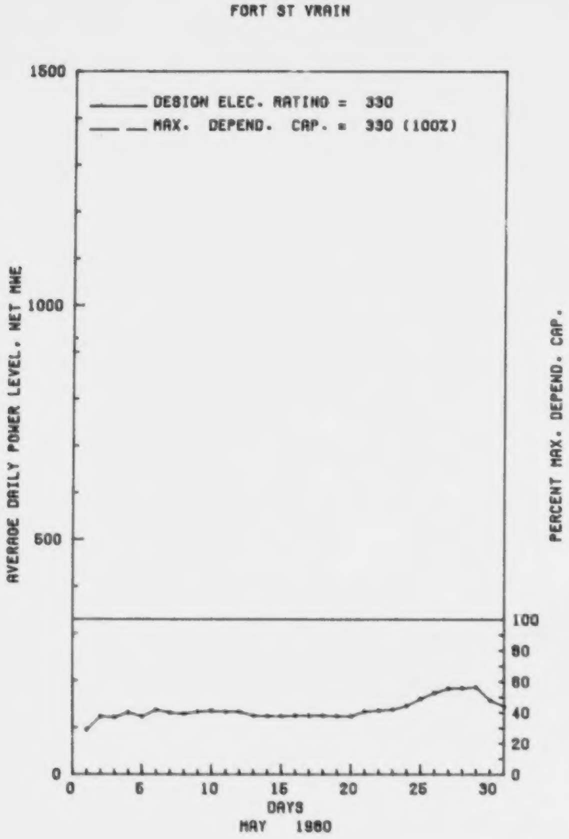
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-19 03L-0	3/27/80	4/25/80	Total oxidants greater than 10 ppm with avg. core outlet temperature greater than 1200 Deg F and primary coolant dewpoint greater than allowed by Technical Specification. Degraded modes of T/S L.C.O. 4.2.10 and 4.2.11, respectively.
80-20 03L-0	4/18/80	5/15/80	Two of twelve Helium Circulator Seal malfunction pressure differential switches tripped outside of L.C.O. 4.4.1 limits
80-21 03L-0	4/21/80	5/21/80	One channel of plant protective system's low Main Steam pressure scram found inoperable. Degraded mode of L.C.O. 4.4.1.
80-22 03L-0	4/24/80	5/22/80	Total oxidants greater than 10 ppm with average core outlet temperature greater than 1200 Deg F. Degraded mode of L.C.O. 4.2.10.

REVIEWED BY OMPA LOEB: T. C. Cintula **DATE:** 7· 3· 80

REVIEWED BY NRR DOR: Gary Kuzmycz **DATE:** 7· 3· 80

FORT ST. VRAIN

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-267
2. Utility Contact: J. W. GAHM (303) 785-2253
3. Licensed Thermal Power (MWt): , 842
4. Nameplate Rating (Gross MWe): 403 x 0.85 = 343
5. Design Electrical Rating (Net MWe): 330
6. Maximum Dependable Capacity (Gross MWe): 342
7. Maximum Dependable Capacity (Net MWe): 330
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): 231
10. Reasons for Restrictions, If Any: RESTRICT. PEND. RESOL. OF TEMP. FLUCTUATIONS.

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	3,647.0	8,064.0
12. Hours Reactor Was Critical	744.0	2,304.9	4,837.7
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	737.6	1,646.9	2,629.1
15. Unit Reserve Shutdown Hours	.0	.0	0
16. Gross Thermal Energy (MWH)	318,452	725,062	1,203,017
17. Gross Electrical Energy (MWH)	108,245	237,023	377,819
18. Net Electrical Energy (MWH)	102,377	220,013	343,597
19. Unit Service Factor	99.1	45.2	32.6
20. Unit Availability Factor	99.1	45.2	32.6
21. Unit Capacity Factor (MDC Net)	41.7	18.3	12.9
22. Unit Capacity Factor (DER Net)	41.7	18.3	12.9
23. Unit Forced Outage Rate	.9	21.7	46.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-11	04/30/80	F	6.4	A	9				TURBINE TAKEN OFF LINE TO RECOVER FROM A LOOP SHUTDOWN.

SUMMARY

FOLLOWING A LOOP SHUTDOWN OUTAGE, THIS UNIT OPERATED ROUTINELY AT RESTRICTED POWER.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

GINNA

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	New York/Wayne 15 miles NE of Rochester
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	1520
5.	Design Electrical Rating (MWE Net)	470
6.	Date of Initial Criticality	November 8, 1969
7.	Date of Commercial Operation	July 1970
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Lake Ontario

Utility & Contractor Information

10.	Licensee	Rochester Gas & Electric Corporation
11.	Corporate Address	89 East Avenue Rochester, New York
12.	Corporate Contact	Leon D. White, Jr., Vice President Electric and Steam Production
13.	Architect/Engineer	Gilbert
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Section Chief	H. B. KISTER
19.	IE Resident Inspector	D. JOHNSON
20.	Licensing Project Manager	S. J. Nowicki
21.	Docket Number	50-244
22.	License Number/Date of Issue	DPR-18, 9/19/69
23.	Public Document Room	Rochester Public Library Business and Social Science Division 115 South Avenue Rochester, New York 14604

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

MANAGERIAL ITEMS:

A management meeting was held at the Ginna Nuclear Power Station on May 15, 1980 to discuss the Systematic Assessment of Licensee Performance (SALP).

PLANT STATUS:

Completed the scheduled refueling outage on May 23, 1980 and at 11:30 PM the station was placed back in service.

LAST IE SITE INSPECTION DATE: 5/15/80

INSPECTION REPORT NO: 50-244/80-07

SUMMARY

INSPECTION SUMMARY

50-244/80-03 - Apr 21-25: Routine, unannounced inspection by one regionally-based inspector (32 hrs.) of refueling and outage activities, including preparations for refueling and previous inspection findings. No items of noncompliance were identified.

50-244/80-05 - Apr 21-25: Unannounced physical protection inspection by one regionally-based inspector (28 hrs.). Included a review of Security Plan; Security Program Audit; Security Organization; Protected Area Physical Barriers; Vital Area Physical Barriers; Lighting; Access Control, Identification, Authorization and Badging, Search, Escorting, Vital Areas; Alarm Stations; Detection Aids; Assessment Aids; Communications; Testing and Maintenance; Compensatory Measures; Power Supply, Response, Locks, Keys and Combinations; Records and Reports; Followup on Previous Items of Noncompliance; Followup on IE Bulletin No. 79-16. No items of noncompliance were identified.

REPORTS RECEIVED FROM LICENSEE

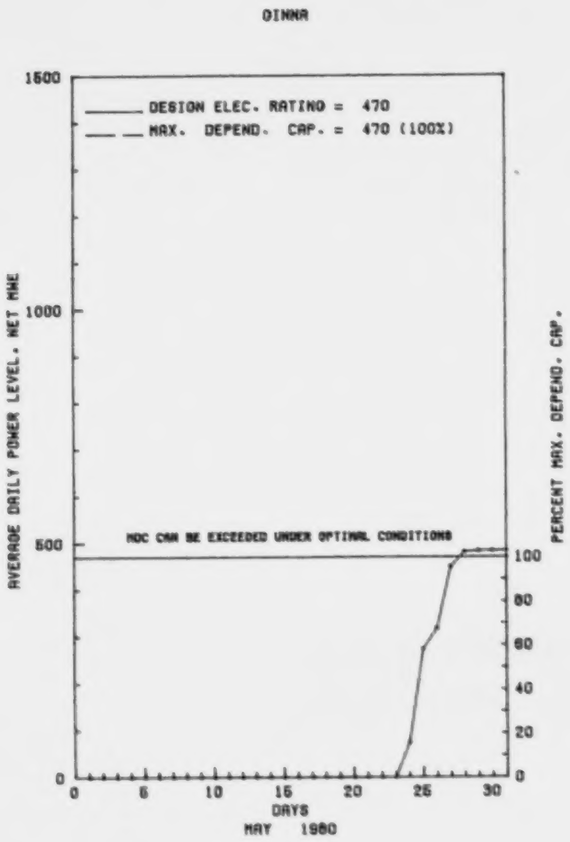
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-003/01T	04/14/80	04/28/80	Results of eddy current testing of steam generators revealed 34 tubes in B generator w/pluggable indications and 1 tube in the A generator
80-004/01T	05/06/80	05/19/80	With the plant in cold shutdown the DC power supply to both PORV's was found to be off rendering both PORV's inoperable; this event is in violation of Tech Spec 3.15.

REVIEWED BY OMPA LOEB: P E Bobe DATE: 7/3/80

REVIEWED BY NRR DOR: T. V. Wambach DATE: 7/8/80

GINNA

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-244
2. Utility Contact: A.E. MCNAMARA (315) 524-4446
3. Licensed Thermal Power (MWt): 1520
4. Nameplate Rating (Gross MWe): 608 x 0.85 = 517
5. Design Electrical Rating (Net MWe): 470
6. Maximum Dependable Capacity (Gross MWe): 490
7. Maximum Dependable Capacity (Net MWe): 470

CAPACITY FACTORS CALCULATED WITH A WEIGHTED AVERAGE

8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	3,647.0	92,159.0
12. Hours Reactor Was Critical	231.5	2,346.0	70,025.8
13. Reactor Reserve Shutdown Hours	.0	.0	1,621.8
14. Hours Generator On-Line	192.8	2,307.3	68,266.1
15. Unit Reserve Shutdown Hours	.0	0	8.5
16. Gross Thermal Energy (MWH)	238,944	3,357,600	91,633,492
17. Gross Electrical Energy (MWH)	77,530	1,098,736	29,784,527
18. Net Electrical Energy (MWH)	73,152	1,043,963	28,205,511
19. Unit Service Factor	25.9	63.3	74.1
20. Unit Availability Factor	25.9	63.3	74.1
21. Unit Capacity Factor (MDC Net)	20.9	60.9	67.5
22. Unit Capacity Factor (DER Net)	20.9	60.9	67.6
23. Unit Forced Outage Rate	.0	.0	9.6

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
SCHEDULED - STEAM GEN. INSPECT. & MODS - 10/24 THRU 11/23.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
1	03/29/80	S	551.2	C	4		RC	FUELXX	ANNUAL REFUELING AND MAINTENANCE SHUTDOWN WHICH BEGAN ON 03/29/80. ENDED ON 05/23/80.

SUMMARY

THE UNIT WAS SHUTDOWN UNTIL 05/23/80 FOR REFUELING. THE REACTOR POWER LEVEL WAS MAINTAINED AT 25% UNTIL 05/24/80 FOR TURBINE OVERSPEED TRIP EAST. LEVEL WAS RAISED TO 100% BY 05/27/80 AND REMAINED THERE UNTIL END OF PERIOD.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

HADDAM NECK

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Connecticut/Middlesex 13 miles E of Meriden
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	1825
5.	Design Electrical Rating (MWE Net)	575
6.	Date of Initial Criticality	July 24, 1967
7.	Date of Commercial Operation	January 1, 1968
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Connecticut River

Utility & Contractor Information

10.	Licensee	Connecticut Yankee Atomic Power Co.
11.	Corporate Address	P. O. Box 270 Hartford, Connecticut
12.	Corporate Contact	D. Switzer, President
13.	Architect/Engineer	Stone & Webster
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	I
17.	IE Section Chief	E. J. Brunner
18.	IE Resident Inspector	R. Keimig
19.	IE Principal Inspector	T. H. Smith
20.	Licensing Project Manager	R. Caruso
21.	Docket Number	50-213
22.	License Number/Date of Issue	DPR-61, 12/27/74
23.	Public Document Room	Russell Library 119 Broad Street Middletown, Connecticut 06457

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS:

Shutdown for refueling. Scheduled outage end date July 5, 1980.

LAST IE SITE INSPECTION DATE: 5/31/80

INSPECTION REPORT NO: 50-213/80-10

SUMMARY

INSPECTION SUMMARY

None

REPORTS RECEIVED FROM LICENSEE

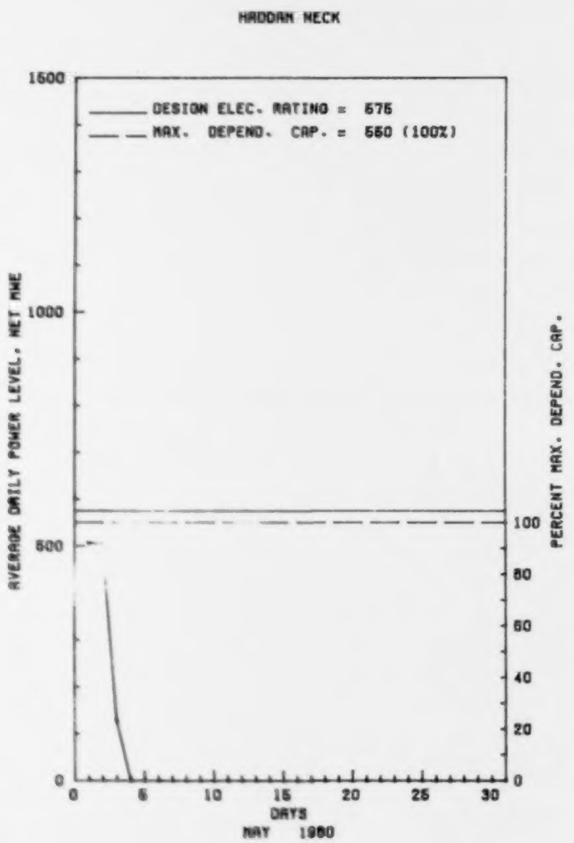
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-002E	05/04/80	05/12/80	TS instantaneous release rate limit exceeded during resin slurrying operation
80-003E	04/28/80	05/12/80	Unplanned release of gaseous radioactivity within TS limits

REVIEWED BY OMPA LOEB: T. C. Cintula DATE: 7·3·80

REVIEWED BY NRR DOR: L. Berger for R. Caruso DATE: 7.7.80

HADDAM NECK

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-213
2. Utility Contact: C. Kokoszka (203) 267-2556
3. Licensed Thermal Power (MWt): 1825
4. Nameplate Rating (Gross MWe): 667 X 0.9 = 600
5. Design Electrical Rating (Net MWe): 575
6. Maximum Dependable Capacity (Gross MWe): 577
7. Maximum Dependable Capacity (Net MWe): 550

CAPACITY FACTORS CALCULATED WITH A WEIGHTED AVERAGE

8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): 88%
10. Reasons for Restrictions, If Any:
TURBINE DISK CRACKING POSSIBILITIES

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	3,647.0	108,839.0
12. Hours Reactor Was Critical	65.5	2,957.1	94,153.8
13. Reactor Reserve Shutdown Hours	.0	11.6	1,164.0
14. Hours Generator On-Line	58.4	2,947.7	89,887.9
15. Unit Reserve Shutdown Hours	.0	.0	369.9
16. Gross Thermal Energy (MWH)	85,371	5,090,070	155,231,219
17. Gross Electrical Energy (MWH)	28,835	1,692,421	51,046,887
18. Net Electrical Energy (MWH)	25,075	1,611,287	48,566,050
19. Unit Service Factor	7.8	80.8	82.6
20. Unit Availability Factor	7.8	80.8	82.9
21. Unit Capacity Factor (MDC Net)	6.1	80.3	82.6
22. Unit Capacity Factor (DER Net)	5.9	76.8	78.1
23. Unit Forced Outage Rate	.0	.5	7.1

24. Shutdowns Scheduled Over Next 6 Months (Type,Date,and Duration of Each):
REFUELING, MAY 3, 1980, APPROX 8 TO 12 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: 07/04/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type¹	Duration (Hours)	Reason²	Method of Shutting Down Reactor³	Licensee Event Report No.	System Code⁴	Component Code⁵	Cause & Corrective Action to Prevent Recurrence
13	05/03/80	S	685.6	C	1		ZZ	ZZZZZZ	TRIPPED TURBINE. NORMAL PLANT UNLOADING FROM 88% POWER TO ZERO FOR REFUELING OUTAGE.

SUMMARY

THE UNIT WAS RUNNING NORMALLY UNTIL 05/03/80, WHEN IT WAS SHUTDOWN FOR REFUELING OUTAGE.

LEGEND

¹Type:
- F--Forced
- S--Scheduled

²Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

³Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

⁴Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

⁵Exhibit H: Same source.

NRC POWER RESTRICTIONS: NONE

HATCH 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Georgia/Appling 11 miles N of Baxley
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2436
5.	Design Electrical Rating (MWE Net)	777
6.	Date of Initial Criticality	September 12, 1974
7.	Date of Commercial Operation	December 31, 1975
8.	Condenser Cooling Method	Once Through With Forced Draft Cooling Towers
9.	Condenser Cooling Water Source	Altamaha River

Utility & Contractor Information

10.	Licensee	Georgia Power Company
11.	Corporate Address	270 Peachtree Street, NE Atlanta, Georgia 30303
12.	Corporate Contact	J. H. Miller Executive Vice President
13.	Architect/Engineer	Southern Services, Inc.
14.	Nuclear Steam Supply System	GE
15.	Constructor	Georgia Power Company

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. Lewis
18.	IE Section Chief	H. C. Dance
19.	IE Resident Inspector	R. F. Rogers (912) 367-9881
20.	Licensing Project Manager	D. Verrelli
21.	Docket Number	50-321
22.	License Number/Date of Issue	DPR-57, 8/6/74
23.	Public Document Room	Appling County Public Library Parker Street Baxley, Georgia 31513

INSPECTION STATUS

ENFORCEMENT STATUS

FAILURE TO FOLLOW PROCEDURES FOR DOCUMENTING SECURITY INCIDENTS.
(8003 INFRACTION)

A VITAL AREA BARRIER DID NOT MEET THE LICENSEE'S SPECIFICATIONS.
(8003 INFRACTION)

ACCESS CONTROL VITAL AREA.
(8003 INFRACTION)

AS REQUIRED BY TECHNICAL SPECIFICATION 4.5.E.2, THE HIGH PRESSURE COOLANT INJECTION SYSTEM (HPCI) IS TO BE PROVEN OPERABLE IMMEDIATELY WHEN THE REACTOR CORE ISOLATION COOLANT SYSTEM (RCIC) IS FOUND TO BE INOPERABLE. TECHNICAL SPECIFICATION 3.5.E.3 REQUIRES THAT THE REACTOR BE SHUTDOWN AND DEPRESSURIZED WITHIN 24 HOURS IF HPCI IS NOT PROVEN OPERABLE. AND AS REPORTED BY THE LICENSEE, HPCI WAS NOT PROVEN OPERABLE FOR APPROXIMATELY 25 HOURS FOLLOWING A RCIC SYSTEM FAILURE ON DECEMBER 8, 1979. PLANT OPERATION CONTINUED THROUGHOUT THIS INTERVAL. THE FAILURE TO TAKE REQUIRED ACTIONS WAS DUE TO PERSONNEL ERROR.
(8004 INFRACTION)

FAILURE TO MAINTAIN LIGHTING EQUIPMENT AS COMMITTED TO IN THE SECURITY PLAN.
(8016 DEFICIENCY)

FAILURE TO PROPERLY CONTROL ALL POINTS OF PERSONNEL ACCESS INTO VITAL AREAS AND TO VITAL EQUIPMENT.
(8016 INFRACTION)

FAILED TO TAKE CORRECTIVE ACTION TO PREVENT REPETITIVE VIOLATIONS OF SECONDARY CONTAINMENT.
(8017 INFRACTION)

FAILURE TO MAKE 24 HOUR AND 30 DAY REPORTS TO THE NRC AS REQUIRED BY TECHNICAL SPECIFICATIONS.
(8017 INFRACTION)

AS REQUIRED BY TECHNICAL SPECIFICATION 6.5.2.8.E, AUDITS OF UNIT ACTIVITIES, INCLUDING THE EMERGENCY PLAN AND IMPLEMENTING PROCEDURES AT LEAST ONCE PER 24 MONTHS, SHALL BE PERFORMED UNDER THE COGNIZANCE OF THE SAFETY REVIEW BOARD. APPENDIX F OF THE EMERGENCY PLAN DEFINES THE EMERGENCY PROCEDURES THAT HAVE BEEN ESTABLISHED TO IMPLEMENT THE EMERGENCY PLAN. AN AUDIT OF THE EMERGENCY IMPLEMENTING PROCEDURES WAS NOT CONDUCTED UNDER THE COGNIZANCE OF THE SAFETY REVIEW BOARD DURING THE PERIOD OCTOBER 1977 TO APRIL 1980.
(8019 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

PLANT IS OPERATING AT FULL POWER.

LAST IE SITE INSPECTION DATE: APRIL 12 - MAY 12, 1980

INSPECTION REPORT NO: 50-321/80-22

REVIEWED BY OMPA LOEB: Earl Hill **DATE:** 8/3/80
REVIEWED BY NRR DOR: **DATE:** 7-7-80

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-023/03L-0	03/08/80	03/27/80	LIQUID RADWASTE RELEASE TO RIVER WITHOUT PROPER ANALYSIS.
80-027/03L-0	03/10/80	04/01/80	ROD BLOCK MONITOR FUNCTIONAL TEST AND CALIBRATION NOT PERFORMED AT REQUIRED INTERVAL.
80-028/03L-0	03/13/80	04/01/80	SECONDARY CONTAINMENT VIOLATED.
80-029/03L-0	03/22/80	04/15/80	DIESEL GENERATOR FAILED TO START AND REACH RATED SPEED.
80-030/03L-0	03/30/80	04/17/80	B HYDROGEN-OXYGEN ANALYZER FOUND INOPERATIVE.
80-031/03L-0	03/30/80	04/17/80	MAIN STACK RADIATION MONITOR PUMP FAILED.
80-033/03L-0	04/02/80	04/24/80	WATERTIGHT SEAL MISSING BETWEEN SALB AND RETAINING WALL.
80-034/03L-0	04/03/80	04/22/80	FIRE WATER TANKS LEVEL BELOW TECHNICAL SPECIFICATION VALVE.
80-035/03L-0	04/07/80	04/23/80	FLOW SWITCH FOR CONTROL ROOM VENTILATION OUT OF CALIBRATION.
80-036/03L-0	04/09/80	04/30/80	FIRE WATER STORAGE TANK BELOW TECHNICAL SPECIFICATION VALUE.
80-037/03L-0	04/08/80	04/29/80	POST LOCA MONITOR CALIBRATION WAS MISSED.
80-038/03L-0	03/27/80	04/18/80	REACTOR MOTOR GENERATOR BREAKER WOULD NOT RESET.
80-039/01T-0	04/14/80	04/22/80	PLANT SERVICE WATER PUMP AND RESIDUAL HEAT REMOVAL SERVICE WATER PUMP DESIGN DEFICIENCY.

SUMMARY

INSPECTION SUMMARY

INSPECTION MARCH 8 - APRIL 11 (80-17): THIS INSPECTION INVOLVED 45 INSPECTOR-HOURS ON SITE IN THE AREAS OF TECHNICAL SPECIFICATION COMPLIANCE, REPORTABLE OCCURRENCES, HOUSEKEEPING, OPERATOR PERFORMANCE, OVERALL PLANT OPERATIONS, QUALITY ASSURANCE PRACTICES, STATION AND CORPORATE MANAGEMENT PRACTICES, CORRECTIVE AND PREVENTATIVE MAINTENANCE ACTIVITIES, SITE SECURITY PROCEDURES, RADIATION CONTROL ACTIVITIES AND SURVEILLANCE ACTIVITIES. OF THE ELEVEN AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN NINE AREAS; TWO ITEMS OF NONCOMPLIANCE WERE FOUND IN TWO AREAS (INFRACTION - FAILURE TO TAKE EFFECTIVE CORRECTIVE ACTION ON SECONDARY CONTAINMENT DOORS (THIS IS A REPEAT ITEM); AND INFRACTION - FAILURE TO REPORT TO THE NRC).

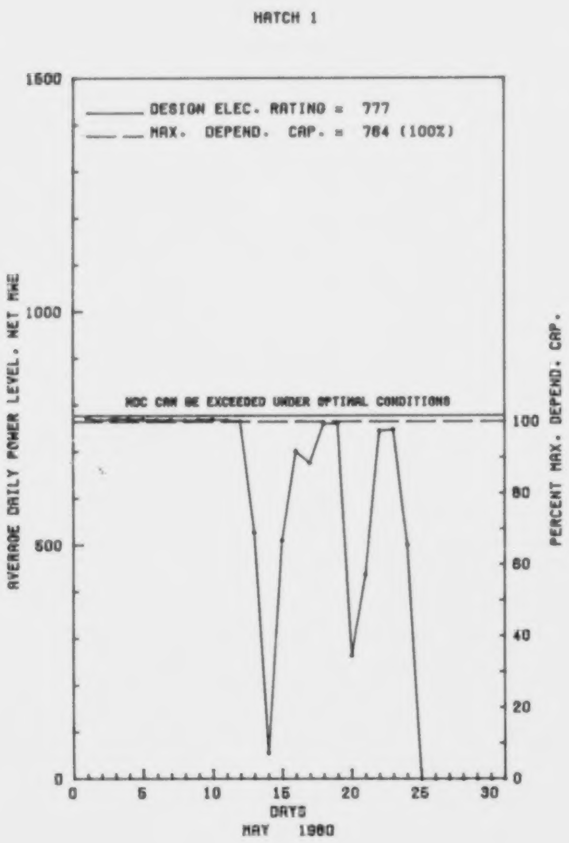
INSPECTION APRIL 21-25 (80-19): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 24 INSPECTOR-HOURS ON SITE IN THE AREAS OF COORDINATION WITH OUTSIDE SUPPORT AGENCIES; EMERGENCY FACILITIES, EQUIPMENT AND PROCEDURES; EMERGENCY TRAINING; EMERGENCY DRILLS; EMERGENCY PLANNING AUDITS; FOLLOWUP ON IE BULLETINS AND FOLLOWUP ON PREVIOUSLY IDENTIFIED INSPECTION FINDINGS. OF THE SEVEN AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN SIX AREAS; ONE ITEM OF NONCOMPLIANCE WAS FOUND IN ONE AREA (INFRACTION - FAILURE TO AUDIT THE EMERGENCY IMPLEMENTING PROCEDURES EVERY TWO YEARS UNDER THE COGNIZANCE OF THE SAFETY REVIEW BOARD).

INSPECTION APRIL 15-18 (80-21): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 12 INSPECTOR-HOURS ON SITE IN THE AREAS OF SURVEILLANCE ACTIVITY ON PIPE SUPPORT AND RESTRAINT SYSTEMS, INCLUDING EXAMINATION OF PROCEDURES TEST RESULTS AND INSTALLED RESTRAINTS, AND EXAMINATION OF RECORDS. OF THE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION APRIL 12 - MAY 12 (80-22): THIS INSPECTION INVOLVED 47 INSPECTOR-HOURS ON SITE IN THE AREAS OF TECHNICAL SPECIFICATION COMPLIANCE, REPORTABLE OCCURRENCES, HOUSEKEEPING, OPERATOR PERFORMANCE, OVERALL PLANT OPERATIONS, QUALITY ASSURANCE PRACTICES, STATION AND CORPORATE MANAGEMENT PRACTICES, CORRECTIVE AND PREVENTATIVE MAINTENANCE ACTIVITIES, SITE SECURITY PROCEDURES, RADIATION CONTROL ACTIVITIES, IE BULLETINS AND SURVEILLANCE ACTIVITIES. OF THE TWELVE AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

HATCH 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-321
2. Utility Contact: J. Roden (912) 367-7781
3. Licensed Thermal Power (MWt): 2436
4. Nameplate Rating (Gross MWe): 1000 x 0.85 = 850
5. Design Electrical Rating (Net MWe): 777
6. Maximum Dependable Capacity (Gross MWe): 797
7. Maximum Dependable Capacity (Net MWe): 764
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	3,647.0	38,711.0
12. Hours Reactor Was Critical	555.3	3,239.1	29,037.1
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	542.3	3,151.1	27,419.5
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,208,270	7,148,840	57,501,975
17. Gross Electrical Energy (MWH)	398,760	2,356,620	18,627,560
18. Net Electrical Energy (MWH)	379,910	2,271,293	17,733,226
19. Unit Service Factor	72.9	86.4	70.8
20. Unit Availability Factor	72.9	86.4	70.8
21. Unit Capacity Factor (MDC Net)	66.8	81.5	60.0
22. Unit Capacity Factor (DER Net)	65.7	80.2	59.0
23. Unit Forced Outage Rate	27.1	13.6	15.8

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUEL-TORUS WORK, 12/06/80, 12 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: 06/09/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-13	05/13/80	F	20.5	A	3		CD	ZZZZZZ	RX SCRAM ON MSIV NOT FULL OPEN WHILE PERFORMING MSIV EXERCISE.
80-14	05/14/80	F	.0	H	5		MB	ZZZZZZ	HOLDING LOAD DUE TO HIGH AIRBORNE ACTIVITY.
80-15	05/17/80	S	.0	B	5		ZZ	CONROD	REDUCTION TO PERFORM WEEKLY TURBINE TEST AND DO ROD PATTERN ADJUSTMENTS.
80-16	05/20/80	F	8.1	A	3		EC	XXXXXX	RX SCRAM ON EHC DC LOSS.
80-17	05/20/80	F	.5	A	1		HC	ZZZZZZ	GEN OFF LINE DUE TO LOSS OF CONDENSER VACUUM THEN ON LINE AT REDUCED LOAD.
80-18	05/24/80	F	172.6	A	3	1-80-53	CF	VALVEX	SHUTDOWN DUE TO INOP. RHR VALVE E11-F017.

SUMMARY

THE REACTOR OPERATED AT FULL POWER UNTIL 05/13/80, WHEN THE UNIT SCRAMMED WHILE TESTING THE MSIV. AFTER RETURNING TO SERVICE, POWER WAS REDUCED DUE TO HIGH AIR ACTIVITY. LOAD WAS REDUCED ON 05/17/80 FOR ROD ADJUSTMENT. THE REACTOR RAN NORMALLY, EXCEPT FOR A LOSS OF DC POWER SCRAM ON THE 20TH, UNTIL 05/24/80, WHEN THE UNIT WAS SHUTDOWN FOR RHR VALVE REPAIR.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

HATCH 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Georgia/Appling 11 miles N of Baxley
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2436
5.	Design Electrical Rating (MWE Net)	784
6.	Date of Initial Criticality	July 4, 1978
7.	Date of Commercial Operation	September 5, 1979
8.	Condenser Cooling Method	Cooling Towers
9.	Condenser Cooling Water Source	Altamaha River

Utility & Contractor Information

10.	Licensee	Georgia Power Company
11.	Corporate Address	270 Peach Street, NE Atlanta, Georgia 30303
12.	Corporate Contact	J. H. Miller Executive Vice President
13.	Architect/Engineer	Southern Services, Inc./Bechtel
14.	Nuclear Steam Supply System	GE
15.	Constructor	Georgia Power Company

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. Lewis
18.	IE Section Chief	H. C. Dance
19.	IE Resident Inspector	R. F. Rogers (912) 367-9881
20.	Licensing Project Manager	D. Verrelli
21.	Docket Number	50-366
22.	License Number/Date of Issue	NPF-5/June 13, 1978
23.	Public Document Room	Appling County Public Library Parker Street Baxley, Georgia 31513

INSPECTION STATUS

ENFORCEMENT STATUS

AS REQUIRED BY TECHNICAL SPECIFICATION 3.1.4.2.B, ONCE CONTROL ROD WITHDRAWAL HAS STARTED AND THERMAL POWER IS LESS THAN 20% OF RATED POWER, CONTROL ROD MOVEMENT IS NOT PERMITTED EXCEPT BY SCRAM WITH THE ROD SEQUENCE CONTROL SYSTEM (RSCS) INOPERABLE. CONTRARY TO THE ABOVE, ON JANUARY 29, 1980, AFTER AN OUT OF SEQUENCE SITUATION WAS DISCOVERED AT APPROXIMATELY 10% POWER, THE LICENSEE BYPASSED ALL GROUP A 3/4 RODS FULL-IN MAKING THE PORTION OF THE RSCS SYSTEM INOPERABLE IN ORDER TO WITHDRAW A GROUP 2 ROD WHICH HAD BEEN ERRONEOUSLY LEFT INSERTED. ALSO ON JANUARY 29, 1980, A REACTOR STARTUP CONTINUED FROM GROUP 5 TO GROUP 7 CONTROL RODS WHILE IT WAS KNOWN TO THE OPERATOR THAT THE GROUP NOTCH CONTROL MODE OF RSCS WAS INOPERATIVE.
(8011 INFRACTION)

AS REQUIRED BY TS 6.8.2, CHANGES TO PLANT PROCEDURES SHALL BE REVIEWED BY THE PLANT REVIEW BOARD (PRB) AND APPROVED BY THE PLANT MANAGER PRIOR TO IMPLEMENTATION. TS 6.8.3 ALLOWS TEMPORARY CHANGES TO PROCEDURES PROVIDED THE INTENT OF THE ORIGINAL PROCEDURE IS NOT ALTERED, THE CHANGE IS DOCUMENTED, REVIEWED BY THE PRB AND APPROVED BY THE PLANT MANAGER WITHIN 14 DAYS. CONTRARY TO THE ABOVE, ON JANUARY 29, 1980 THE LICENSEE RESTORED TWO OUT OF SEQUENCE RODS TO THEIR REQUIRED POSITIONS WHILE AT 10% POWER WHICH CHANGED THE INTENT OF HNP-2-9207, ROD MOVEMENT SEQUENCES, WHICH REQUIRES ROD WITHDRAWAL IN A PREDETERMINED, CONSECUTIVE ORDER. THIS CHANGE WAS NOT DOCUMENTED AS A TEMPORARY CHANGE OR A CHANGE REQUIRING PRIOR APPROVAL.
(8011 INFRACTION)

AS REQUIRED BY TS 6.9.1.8.F, ANY PERSONNEL ERROR OR PROCEDURAL INADEQUACY WHICH PREVENTS OR COULD PREVENT, BY ITSELF, THE FULFILLMENT OF THE FUNCTIONAL REQUIREMENTS OF SYSTEMS REQUIRED TO COPE WITH ACCIDENTS ANALYZED IN THE SAR IS REQUIRED TO BE REPORTED WITHIN 24 HOURS TO THE DIRECTOR OF THE REGIONAL OFFICE. CONTRARY TO THE ABOVE, ON JANUARY 29, 1980, THE EXISTENCE OF AN OUT OF SEQUENCE ROD PATTERN INVOLVING TWO CONTROL RODS AT APPROXIMATELY 10% POWER WHICH NEITHER THE ROD WORTH MINIMIZER, THE ROD SEQUENCE CONTROL SYSTEM, PROCEDURAL CONTROLS, NOR OPERATOR ALERTNESS PREVENTED, WAS NOT REPORTED BY THE LICENSEE. THE ABOVE TWO SAFETY SYSTEMS IMPOSE RESTRICTIONS ON ROD PATTERNS TO MITIGATE A ROD DROP ACCIDENT.
(8011 INFRACTION)

FAILURE TO MAINTAIN LIGHTING EQUIPMENT AS COMMITTED TO IN THE SECURITY PLAN.
(8016 DEFICIENCY)

FAILURE TO PROPERLY CONTROL ALL POINTS OF PERSONNEL ACCESS INTO VITAL AREAS AND TO VITAL EQUIPMENT.
(8016 INFRACTION)

FAILED TO TAKE CORRECTIVE ACTION TO PREVENT REPETITIVE VIOLATIONS OF SECONDARY CONTAINMENT.
(8017 INFRACTION)

FAILURE TO MAKE 24 HOUR AND 30 DAY REPORTS TO THE NRC AS REQUIRED BY TECHNICAL SPECIFICATIONS.
(8017 INFRACTION)

CONTRARY TO TS 6.8.1, THE LICENSEE'S TESTS OF THE MODIFIED RSCS WERE INADEQUATE TO REVEAL INSTALLATION ERRORS WHICH RENDERED THE RSCS INOPERABLE.
(8018 INFRACTION)

AS REQUIRED BY TECHNICAL SPECIFICATION 6.5.2.8.E, AUDITS OF UNIT ACTIVITIES, INCLUDING THE EMERGENCY PLAN AND IMPLEMENTING PROCEDURES AT LEAST ONCE PER 24 MONTHS, SHALL BE PERFORMED UNDER THE COGNIZANCE OF THE SAFETY REVIEW BOARD. APPENDIX F OF THE EMERGENCY PLAN DEFINES THE EMERGENCY PROCEDURES THAT HAVE BEEN ESTABLISHED TO IMPLEMENT THE EMERGENCY PLAN. AN AUDIT OF THE EMERGENCY IMPLEMENTING PROCEDURES WAS NOT CONDUCTED UNDER THE COGNIZANCE OF THE SAFETY REVIEW BOARD DURING THE PERIOD OCTOBER 1977 TO APRIL 1980.
(8019 INFRACTION)

SEE CONTINUATION PAGE H-7A.

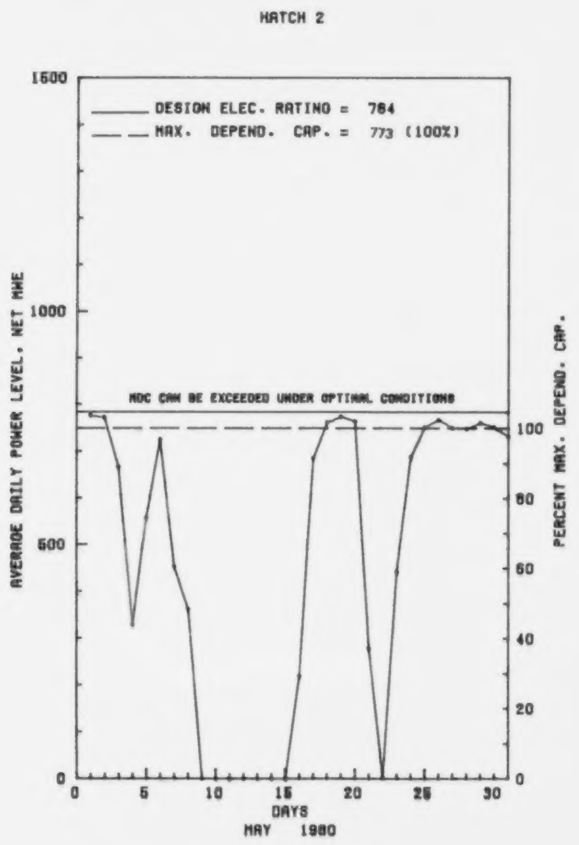
REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-021/ 03L-0	03/03/80	03/26/80	CONTAINMENT ISOLATION VALVES FAILED LEAKAGE TEST.
80-024/ 03L-0	03/08/80	03/31/80	RESIDUAL HEAT REMOVAL SERVICE WATER PUMP INDICATED LOW FLOW DURING SURVEILLANCE TEST.
80-026/ 03L-0	03/14/80	04/02/80	LEAK RATE TEST SHOWED HYDROGEN RECOMBINER ISOLATION VALVES LEAKED EXCESSIVELY.
80-032/ 01T-0	03/27/80	04/07/80	PLANT SERVICE WATER PUMP DID NOT MEET TIME SEQUENCE TO EMERGENCY BUS DURING TEST.
80-033/ 03L-0	03/31/80	04/15/80	TRIAXIAL PEAK RECORDING ACCELEROMETERS ON REACTOR PEDESTAL AND FEEDWATER LINE NOT CALIBRATED AT REQUIRED INTERVAL.
80-034/ 03L-0	03/20/80	04/15/80	MAIN STEAM LINE TEMPERATURE SWITCHES OUT OF CALIBRATION.
80-035/ 03L-0	03/21/80	04/17/80	ACCUMULATOR PRESSURE SWITCHES OUT OF SPECIFICATION.
80-036/ 03L-0	03/26/80	04/22/80	VALVE 2T23-F003 FAILED LEAK RATE TESTING.
80-037/ 03L-0	03/28/80	04/17/80	REACTOR WATER CHEMISTRY CHLORIDE CONCENTRATION OUT OF SPECIFICATION.
80-038/ 03L-0	03/28/80	04/23/80	DRYWELL TO TORUS ISOLATION VALVE FAILED FUNCTIONAL TEST.
80-039/ 01T-1	04/11/80	04/30/80	PLANT SERVICE WATER PUMP MOTORS COOLING WATER SUPPLY DOES NOT MEET SINGLE FAILURE CRITERION.
80-039/ 03L-0	03/25/80	04/15/80	NO FLOW ON STAND BY GAS TREATMENT VENTILATION.
80-040/ 03L-0	03/31/80	04/16/80	SURVEILLANCES NOT PERFORMED AT REQUIRED TIME.
80-041/ 03L-01	04/01/80	04/21/80	VALVE 2B21-F077A FAILED LEAK RATE TESTING.
80-042/ 03L-0	03/28/80	04/22/80	HYDRAULIC SNUBBER FAILED TESTING.
80-043/ 03L-0	04/01/80	04/17/80	CARBON FOUND ON FLOOR DURING FILTER INSPECTION VENTILATION SYSTEM.
80-044/ 03L-0	04/02/80	04/17/80	DRYWELL TEMPERATURE SENSOR AND SUPPRESSION POOL TEMPERATURE INDICATOR NOT CONNECTED CORRECTLY.
80-045	04/06/80	04/15/80	AUTOMATIC DEPRESSURIZATION SYSTEM CHECK VALVES EXCESSIVE SEAT LEAKAGE.
80-046/ 03L-0	04/04/80	04/28/80	SURVEILLANCE TEST ON DIESEL GENERATOR RESULTED IN IMPROPER SYNCHRO SETTING.
80-047/ 03L-0	04/05/80	04/29/80	TORUS WATER LEVEL INSTRUMENT TRANSMITTER IMPROPERLY INSTALLED.
80-048/ 03L-0	04/15/80	04/29/80	LOOSE CONNECTION CAUSE OVERHEATING IN OVERCURRENT RELAY CIRCUIT AT TERMINAL BLOCK.
80-049/ 03L-0	04/07/80	04/22/80	STANDBY LIQUID CONTROL PUMP RELIEF VALVE FAILED LIFT TEST.
80-051/ 03L-0	03/09/80	04/30/80	RADWASTE ISOLATION VALVE FAILED SEAT LEAKAGE TEST.
80-053	04/16/80	04/23/80	RESIDUAL HEAT REMOVAL PUMP LOGIC ERROR.
80-056/ 03L/1	04/14/80	05/05/80	REACTOR PROTECTION SYSTEM-GENERATOR "A" LOGIC POWER FAILURE.
80-058/ 0T-1	04/17/80	04/30/80	BREAKER TO EMERGENCY BUS FROM DIESEL GENERATOR WOULD NOT OPEN WHEN DIESEL SHUTDOWN.
80-059/ 03L-0	04/16/80	05/06/80	OUTBOARD MSIV HAD NON-QUALIFIED LIMIT SWITCH REQUIRED QUALIFIED SWITCH.

- CONTINUED -

HATCH-2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-366
2. Utility Contact: J. Roden (912) 367-7781
3. Licensed Thermal Power (MWt): 2436
4. Nameplate Rating (Gross MWe): 1000 x .85 = 850
5. Design Electrical Rating (Net MWe): 784
6. Maximum Dependable Capacity (Gross MWe): 806
7. Maximum Dependable Capacity (Net MWe): 773
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	3,647.0	6,480.0
12. Hours Reactor Was Critical	594.9	2,208.7	4,872.2
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	534.3	2,043.1	4,457.8
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,113,360	4,435,150	9,929,212
17. Gross Electrical Energy (MWH)	364,520	1,479,620	3,315,580
18. Net Electrical Energy (MWH)	345,876	1,407,133	3,164,270
19. Unit Service Factor	71.8	56.0	68.8
20. Unit Availability Factor	71.8	56.0	68.8
21. Unit Capacity Factor (MDC Net)	60.2	49.9	63.2
22. Unit Capacity Factor (DER Net)	59.3	49.2	62.3
23. Unit Forced Outage Rate	28.2	12.8	10.0

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUEL-TORUS WORK, 10/11/80, 8-WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type¹	Duration (Hours)	Reason²	Method of Shutting Down Reactor³	Licensee Event Report No.	System Code⁴	Component Code⁵	Cause & Corrective Action to Prevent Recurrence
80-10	05/03/80	S	.0	H	5		ZZ	CONROD	REDUCTION TO PERFORM WEEKLY TURBINE TEST AND DO CONTROL ROD PATTERN ADJUSTMENT.
80-11	05/04/80	F	.0	B	5		CH	PIPEXX	REDUCTION TO REPAIR BROKEN DRAIN LINE ON RFPT.
80-12	05/06/80	F	.0	B	5		CB	PIPEXX	REDUCTION TO REPAIR OIL LEAK ON MG SET "A".
80-13	05/08/80	F	174.1	A	1		HA	XXXXXX	SHUTDOWN DUE TO GROUND FAULT IN #8 TURB-GEN BEARING.
80-14	05/16/80	F	.0	B	5		HH	PIPEXX	REDUCTION TO REPAIR LEAKING DRAIN LINE ON CONDENSATE BOOSTER PUMP.
80-15	05/21/80	F	23.7	H	3		SH	ZZZZZZ	RX SCRAM ON FALSE LOW LEVEL DUE TO TEST SHOP ISOLATING WRONG VALVE.
80-16	05/22/80	F	11.9	A	2		HJ	VALVES	GEN OFF LINE DUE TO 4TH STG. EXTRACTION MOV NOT OPERATING.
80-17	05/23/80	F	.0	H	5		ZZ	ZZZZZZ	REDUCTION DUE TO MFLPD, PCIOMR, HEAT RATE AND TURBINE TESTING.

SUMMARY

THE UNIT RAN NEAR FULL POWER, WITH SCRAMS FROM 05/08/80 TO 05/15/80 DUE TO A BEARING FAULT, AND ON 05/21/80 BECAUSE OF A FALSE LOW WATER LEVEL SIGNAL. REDUCTIONS OCCURRED ON THE 4TH, 6TH AND 10TH FOR DRAIN LINE REPAIRS.

LEGEND

¹Type:
F--Forced
S--Scheduled

²Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

³Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

⁴Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

⁵Exhibit H:
Same source.

CONTINUATION PAGE

HATCH 2

ENFORCEMENT STATUS

FAILURE TO FOLLOW PROCEDURES FOR DOCUMENTING SECURITY INCIDENTS.
(8003 INFRACTION)

A VITAL AREA BARRIER DID NOT MEET THE LICENSEE'S SPECIFICATIONS.
(8003 INFRACTION)

ACCESS CONTROL - VITAL AREA.
(8003 INFRACTION)

AS REQUIRED BY CRITERION VI OF APPENDIX B TO 10 CFR 50 AND AS DESCRIBED IN THE FINAL SAFETY ANALYSIS REPORT, SECTION 17.2.6.3, MEASURES SHALL BE ESTABLISHED TO CONTROL THE ISSUANCE OF DRAWING CHANGES WHICH ASSURE THAT THESE CHANGES ARE DISTRIBUTED TO AND USED AT THE LOCATION WHERE THE PRESCRIBED ACTIVITY IS PERFORMED. CONTRARY TO THE ABOVE, ON MARCH 4, 1980, DURING A SPOT CHECK OF UNIT 2 DRAWINGS MAINTAINED IN THE CONTROL ROOM, 16 OF 30 DRAWINGS CHECKED WERE NOT THE CURRENT REVISION AND WERE ROUTINELY BEING USED IN THE CONDUCT OF PLANT OPERATIONS.
(8011 INFRACTION)

AS REQUIRED BY TS 3.1.3.7.A.2, CONTROL RODS MAY BE BYPASSED IN THE ROD SEQUENCE CONTROL SYSTEM IF THE CONTROL ROD REED SWITCH FULL-IN AND/OR FULL-OUT POSITION INDICATORS ARE INOPERABLE. CONTRARY TO THE ABOVE, ON JANUARY 28, 1980, CONTROL RODS 46-43 AND 14-43 WERE BYPASSED FULL OUT WITH OPERABLE REED SWITCH POSITION INDICATORS TO FACILITATE ROD SELECTION DURING A REACTOR STARTUP. ALSO ON JANUARY 29, 1980, ALL GROUP A 3/4 RODS WERE BYPASSED WITH OPERABLE REED SWITCH POSITION INDICATORS TO RESTORE MISPOSITIONED CONTROL ROD 46-43 TO ITS PROPER POSITION IN ORDER TO CONTINUE A PLANT STARTUP.
(8011 INFRACTION)

AS REQUIRED BY TS 3.1.3.7.A.2.B, A CONTROL ROD WHICH IS BYPASSED IN THE ROD SEQUENCE CONTROL SYSTEM IS REQUIRED TO BE MOVED TO THE CORRECT POSITION IN THE PROPER SEQUENCE IN ORDER TO CONTINUE REACTOR OPERATION. TS 6.8 REQUIRES THAT WRITTEN PROCEDURES SHALL BE ESTABLISHED AND IMPLEMENTED. HATCH NUCLEAR PLANT PROCEDURE HNP-2-9207, ROD MOVEMENT SEQUENCES, REQUIRES THAT CONTROL RODS BE MOVED IN THE PROPER SEQUENCE. CONTRARY TO THE ABOVE, DURING A REACTOR STARTUP COMMENCING ON JANUARY 28, 1980, THE REACTOR WAS TAKEN TO POWER WITH CONTROL ROD 46-43 LEFT FULLY INSERTED AND BYPASSED IN THE ROD SEQUENCE CONTROL SYSTEM RATHER THAN FULLY WITHDRAWN AND CONTROL ROD 10-47 LEFT AT NOTCH 12 RATHER THAN FULLY WITHDRAWN DUE TO PERSONNEL ERRORS.
(8011 INFRACTION)

INSPECTION STATUS

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

PLANT IS OPERATING AT FULL POWER.

LAST IE SITE INSPECTION DATE: APRIL 12 - MAY 12, 1980

INSPECTION REPORT NO: 50-366/80-22

SUMMARY

INSPECTION SUMMARY

INSPECTION MARCH 8 - APRIL 11 (80-17): THIS INSPECTION INVOLVED 45 INSPECTOR-HOURS ON SITE IN THE AREAS OF TECHNICAL SPECIFICATION COMPLIANCE, REPORTABLE OCCURRENCES, HOUSEKEEPING, OPERATOR PERFORMANCE, OVERALL PLANT OPERATIONS, QUALITY ASSURANCE PRACTICES, STATION AND CORPORATE MANAGEMENT PRACTICES, CORRECTIVE AND PREVENTATIVE MAINTENANCE ACTIVITIES, SITE SECURITY PROCEDURES, RADIATION CONTROL ACTIVITIES AND SURVEILLANCE ACTIVITIES. OF THE ELEVEN AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN NINE AREAS; TWO ITEMS OF NONCOMPLIANCE WERE FOUND IN TWO AREAS (INFRACTION - FAILURE TO TAKE EFFECTIVE CORRECTIVE ACTION ON SECONDARY CONTAINMENT DOORS (THIS IS A REPEAT ITEM); AND INFRACTION - FAILURE TO REPORT TO THE NRC).

INSPECTION APRIL 21-25 (80-19): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 24 INSPECTOR-HOURS ON SITE IN THE AREAS OF COORDINATION WITH OUTSIDE SUPPORT AGENCIES; EMERGENCY FACILITIES, EQUIPMENT AND PROCEDURES; EMERGENCY TRAINING; EMERGENCY DRILLS; EMERGENCY PLANNING AUDITS; FOLLOWUP ON IE BULLETINS AND FOLLOWUP ON PREVIOUSLY IDENTIFIED INSPECTION FINDINGS. OF THE SEVEN AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN SIX AREAS; ONE ITEM OF NONCOMPLIANCE WAS FOUND IN ONE AREA (INFRACTION - FAILURE TO AUDIT THE EMERGENCY IMPLEMENTING PROCEDURES EVERY TWO YEARS UNDER THE COGNIZANCE OF THE SAFETY REVIEW BOARD).

INSPECTION APRIL 15-18 (80-21): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 12 INSPECTOR-HOURS ON SITE IN THE AREAS OF SURVEILLANCE ACTIVITY ON PIPE SUPPORT AND RESTRAINT SYSTEMS, INCLUDING EXAMINATION OF PROCEDURES TEST RESULTS AND INSTALLED RESTRAINTS, AND EXAMINATION OF RECORDS. OF THE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION APRIL 12 - MAY 12 (80-22): THIS INSPECTION INVOLVED 47 INSPECTOR-HOURS ON SITE IN THE AREAS OF TECHNICAL SPECIFICATION COMPLIANCE, REPORTABLE OCCURRENCES, HOUSEKEEPING, OPERATOR PERFORMANCE, OVERALL PLANT OPERATIONS, QUALITY ASSURANCE PRACTICES, STATION AND CORPORATE MANAGEMENT PRACTICES, CORRECTIVE AND PREVENTATIVE MAINTENANCE ACTIVITIES, SITE SECURITY PROCEDURES, RADIATION CONTROL ACTIVITIES, IE BULLETINS AND SURVEILLANCE ACTIVITIES. OF THE TWELVE AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

REPORTS FROM LICENSEE

80-063/ 03L-0	04/21/80	05/06/80	DRYWELL TORUS TEMPERATURE RECORDER INOPERATIVE.
80-069/ 03L-0	04/21/80	05/06/80	MINIMUM FLOW CONTROL VALVE BEGAN CYCLING FROM FULL CLOSE TO OPEN HPCI SYSTEM.
80-070/ 03L-0	04/22/80	05/05/80	RCIC TRIP AND THROTTLE VALVE CLOSED.
80-071/ 03L-0	04/24/80	05/06/80	INVERTER SUPPLYING THE POWER TO THE LEAK DETECTION LOGIC FAILED.

REVIEWED BY OMPA LOEB: Early Dell **DATE:** 7/3/80

REVIEWED BY NRR DOR: **DATE:** 7-7-80

HUMBOLDT BAY

FACILITY DATA

Facility Description

1. Location (state/countv) (nearest population center)	California/Humboldt 4 miles S of Eureka
2. Electric Reliability Council	Western Systems Coordinating Council
3. Type of Reactor	BWR
4. Power Level (MWT)	220
5. Design Electrical Rating (MWE Net)	65
6. Date of Initial Criticality	February 16, 1963
7. Date of Commercial Operation	August 1963
8. Condenser Cooling Method	Circulating sea water
9. Condenser Cooling Water Source	Pacific Ocean (Humboldt Bay)

Utility & Contractor Information

10. Licensee	Pacific Gas & Electric Co.
11. Corporate Address	77 Beale Street San Francisco, California 94106
12. Corporate Contact	P. A. Crane, Jr. Asst. General Counsel
13. Architect/Engineer	Bechtel
14. Nuclear Steam Supply System	GE
15. Constructor	Bechtel

Regulatory Information

16. IE Region responsible	V
17. IE Branch Chief	J. L. Crews
18. IE Section Chief	D. M. Sternberg
19. IE Resident Inspector	J. D. Carlson
20. Licensing Project Manager	V. L. Rooney
21. Docket Number	50-133
22. License Number/Date of Issue	DPR-7, 8/28/62
23. Public Document Room	Humboldt County Library 636 F Street Eureka, California 95501

INSPECTION STATUS

ENFORCEMENT STATUS

10 CFR 20.203(C)(1), "CAUTION SIGNS, LABELS, SIGNALS AND CONTROLS," REQUIRES THAT EACH HIGH RADIATION AREA BE CONSPICUOUSLY POSTED WITH A SIGN OR SIGNS BEARING THE RADIATION CAUTION SYMBOL AND THE WORDS "CAUTION - HIGH RADIATION AREA." CONTRARY TO THIS REQUIREMENT, ON MARCH 26, 1980 THE HIGH LEVEL RADIOACTIVE WASTE STORAGE VAULT WAS NOT CONSPICUOUSLY POSTED WITH A SIGN OR SIGNS BEARING THE WORDS: "CAUTION - HIGH RADIATION AREA." THIS MATTER WAS CORRECTED AT THE TIME OF THE INSPECTION.
(8001 INFRACTION)

10 CFR 20.203(C)(2)(III), "CAUTION SIGNS, LABELS, SIGNALS, AND CONTROLS," REQUIRES THAT IN THE ABSENCE OF CERTAIN CONTROL DEVICES EACH ENTRANCE OR ACCESS POINT TO A HIGH RADIATION AREA SHALL BE MAINTAINED LOCKED EXCEPT DURING PERIODS WHEN ACCESS TO THE AREA IS REQUIRED WITH POSITIVE CONTROL OVER EACH INDIVIDUAL ENTRY. (1) CONTRARY TO THIS REQUIREMENT, ON MARCH 26, 1980 THE SOUTH ENTRANCE DOOR TO THE RESIN REGENERATION ROOM WAS UNLOCKED AND POSITIVE CONTROL OVER EACH INDIVIDUAL ENTRY HAD NOT BEEN ESTABLISHED. THE GENERAL AREA DOSE RATE INSIDE THE ROOM WAS MEASURED TO BE 150 MREM/HR. (2) CONTRARY TO THIS REQUIREMENT, ON MARCH 26, 1980 A PERMANENTLY INSTALLED LADDER PROVIDED ACCESS TO A HIGH RADIATION AREA NEAR WASTE RECEIVER TANK NO. 2. THE MEASURED DOSE RATE NEAR THIS TANK WAS 800 MREM/HR.
(8001 INFRACTION)

10 CFR 20.203(F)(1) AND (2), "CAUTION SIGNS, LABELS, SIGNALS AND CONTROLS," REQUIRES THAT EACH CONTAINER OF LICENSED MATERIAL SHALL BEAR A LABEL SHOWING THE RADIATION CUATION SYMBOL, THE WORDS, "CAUTION - RADIOACTIVE MATERIAL," AND IDENTIFICATION OF THE RADICACTIVE CONTENTS. CONTRARY TO THIS REQUIREMENT, ON MARCH 26, 1980, TWO 4' X 4' X 8' WOOD BOXES LOCATED IN THE RESTRICTED AREA NORTH OF THE REFUELING BUILDING CONTAINED AN ESTIMATED TWO MILLICURIES EACH OF LICENSED MATERIALS AND DID NOT BEAR A LABEL.
(8001 DEFICIENCY)

INSPECTION STATUS

SYSTEMS AND COMPONENT PROBLEMS:

None.

FACILITY ITEMS (PLANS AND PROCEDURES):

The plant has been shut down since July 1976.

Several items remaining to be completed during the outage have been deferred pending resolution of the seismic issues at the plant site. A date for return to operations has not been established.

MANAGERIAL ITEMS:

None

PLANT STATUS:

Plant is shutdown.

LAST IE SITE INSPECTION DATE: 05/20-22/80+

INSPECTION REPORT NO: 50-133/80-02+

SUMMARY

INSPECTION SUMMARY

Inspection on May 20-22, 1980 (Report No. 50-133/80-02) Report being prepared; to be reported next month.

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE			

LICENSE STATUS

Order issued 5/21/76 requiring Utility to prove seismic qualification before resuming operation of unit after outage of 7/2/76.

REVIEWED BY OMPA LOEB: John J. Rizzo DATE: 7-3-80

REVIEWED BY NRR DOR: DATE: 7/7/80

HUMBOLDT BAY

AVERAGE DAILY POWER LEVEL (MWe)

OPERATING STATUS

1. Docket Number: 50-133
2. Utility Data Prepared By: B. C. Getty (707) 443-2787
3. Licensed Thermal Power (MWt): 220
4. Nameplate Rating (Gross MWe): 77 X 0.85 = 65
5. Design Electrical Rating (Net MWe): 65
6. Maximum Dependable Capacity (Gross MWe): 0
7. Maximum Dependable Capacity (Net MWe): 0
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons

None

9. Power Level To Which Restricted, If Any (Net MWe): None
10. Reasons For Restrictions, If Any:

This ive

11. Hours In Reporting Period
12. Number Of Hours Reactor Was Cri
13. Reactor Reserve Shutdow
14. Hours Generato
15. Unit R
1 rd)
erated (MWH)
Net gy Generated (MWH)
19. Unit S ce Factor
20. Unit Availability Factor
21. Unit Capacity Factor (Using MDC Net) 1/
22. Unit Capacity Factor (Using DER Net) 1/
23. Unit Forced Outage Rate
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

None

25. If Shut Down At End Of Report Period, Estimated Date of Startup: UNKNOWN

NO DECISION ON FUTURE OPERATION
SHUTDOWN INDEFINITELY

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
SEE SUMMARY BELOW									

SUMMARY

THE UNIT WAS SHUTDOWN ON 7/2/76 FOR SEISMIC MODIFICATIONS. THE UNIT REMAINS IN THE COLD SHUTDOWN CONDITION.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

INDIAN POINT 1

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	New York/Westchester 25 miles N of New York City
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	PWR/fossil superheat
4.	Power Level (MWT)	615
5.	Design Electrical Rating (MWE Net)	265
6.	Date of Initial Criticality	August 2, 1962
7.	Date of Commercial Operation	October 1962
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Hudson River

Utility & Contractor Information

10.	Licensee	Consolidated Edison Company of New York, Inc.
11.	Corporate Address	4 Irving Place New York, New York
12.	Corporate Contact	W. Cahill, Vice President
13.	Architect/Engineer	Utility
14.	Nuclear Steam Supply System	B&W
15.	Constructor	Utility

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Section Chief	H. B. KISTER
19.	IE Resident Inspector	T. Rebelowski (914-739-9360) FTS: 656-9220
20.	Licensing Project Manager	L. Olshan x27144
21.	Docket Number	50-03
22.	License Number/Date of Issue	DPR-5, 3/26/62
23.	Public Document Room	White Plains Public Library 100 Martine Avenue White Plains, New York 10601

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

- MANAGERIAL ITEMS:
- On April 12, 1980, Mr. M. S. Skoszko assumed the duties of Security Administrator.

PLANT STATUS.

Defueled.

LAST IE SITE INSPECTION DATE: 7/24-27/79

INSPECTION REPORT NO: 50-03/79-02

3. SUMMARY

INSPECTION SUMMARY

- None conducted.

LICENSE STATUS

The unit produced electricity to the expiration date (October 31, 1974) of delay granted by the AEC for installation of an ECCS system. The ECCS system was not subsequently installed and the unit was defueled on January 30, 1976.

On February 11, 1980, the NRC requested the utility to show cause as to why the operating authority of the license should not be revoked and why the licensee should not submit a plan within 120 days of this order to decommission the unit.

On June 19, 1980, an order was issued by the NRC to revoke the operating authority for Indian Point Station, Unit 1, as an electrical generating facility. Consolidated Edison Company of New York, Inc. is to submit a decommissioning plan for Unit 1 within 120 days.

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
NONE				

REVIEWED BY OMPA LOEB: T. C. Cintula **DATE:** 7-3-80

REVIEWED BY NRR DOR: **DATE:** 7-7-80

INDIAN POINT 1

AVERAGE DAILY POWER LEVEL (MWe)

OPERATING STATUS

1. Docket Number: 50-003
2. Utility Data Prepared By: L. J. Kawula (914)-694-6000 x209
3. Licensed Thermal Power (MWt): 615
4. Nameplate Rating (Gross MWe): 275
5. Design Electrical Rating (Net MWe): 266
6. Maximum Dependable Capacity (Gross MWe): 0
7. Maximum Dependable Capacity (Net MWe): 0
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons
None
9. Power Level To Which Restricted, If Any (Net MWe): None
10. Reasons For Restrictions, If Any

This Month Yr.-to-

11. Hours In Reporting Period
12. Number Of Hours Reactor Was Critical
13. Reactor Reserve Shutdown Hours
14. Hours Generator On-Line
15. Unit Reserve Shutdown H
16. Gross Thermal E
17. Gross E
1
.or
, Factor (Using MDC Net) 1/
. Capacity Factor (Using DER Net) 1/
23. Unit Forced Outage Rate
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
None
25. If Shut Down At End Of Report Period, Estimated Date of Startup: N/A

UNIT RETIRED AS GENERATING FACILITY.
DECOMMISSIONING PLAN TO BE SUBMITTED
BY LICENSEE

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
SEE SUMMARY BELOW									

SUMMARY

UNIT SHUTDOWN ON OCTOBER 31, 1974, ALL FUEL HAS BEEN UNLOADED; UNIT RETIRED AS A GENERATING FACILITY ON JUNE 19, 1980; DECOMMISSIONING PLAN TO BE SUBMITTED.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

INDIAN POINT 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1. Location (state/county) (nearest population center)	New York/Westchester 25 miles N of New York City
2. Electric Reliability Council	Northeast Power Coordinating Council
3. Type of Reactor	PWR
4. Power Level (MWT)	2758
5. Design Electrical Rating (MWE Net)	873
6. Date of Initial Criticality	May 22, 1973
7. Date of Commercial Operation	August 1973
8. Condenser Cooling Method	Once Through
9. Condenser Cooling Water Source	Hudson River

Utility & Contractor Information

10. Licensee	Consolidated Edison Company of New York, Inc.
11. Corporate Address	4 Irving Place New York, New York
12. Corporate Contact	W. J. Cahill, Jr., Vice President
13. Architect/Engineer	UE&C
14. Nuclear Steam Supply System	Westinghouse
15. Constructor	J. A. Jones/UE&C

Regulatory Information

16. IE Region responsible	I
17. IE Branch Chief	E. J. Brunner
18. IE Senior Inspectc	H. B. KISTER
19. IE Resident Inspector	T. Rebelowski (914-739-9360) FTS: 656-9220
20. Licensing Project Manager	L. Olshan (X27144)
21. Docket Number	50-247
22. License Number/Date of Issue	DPR-26, 10/19/71
23. Public Document Room	White Plains Public Library 100 Martine Avenue White Plains, New York 10601

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS:

Unit at 100% power.

LAST IE SITE INSPECTION DATE: 5/23/80

INSPECTION REPORT NO: 50-247/80-08

SUMMARY

INSPECTION SUMMARY

None.

REPORTS RECEIVED FROM LICENSEE

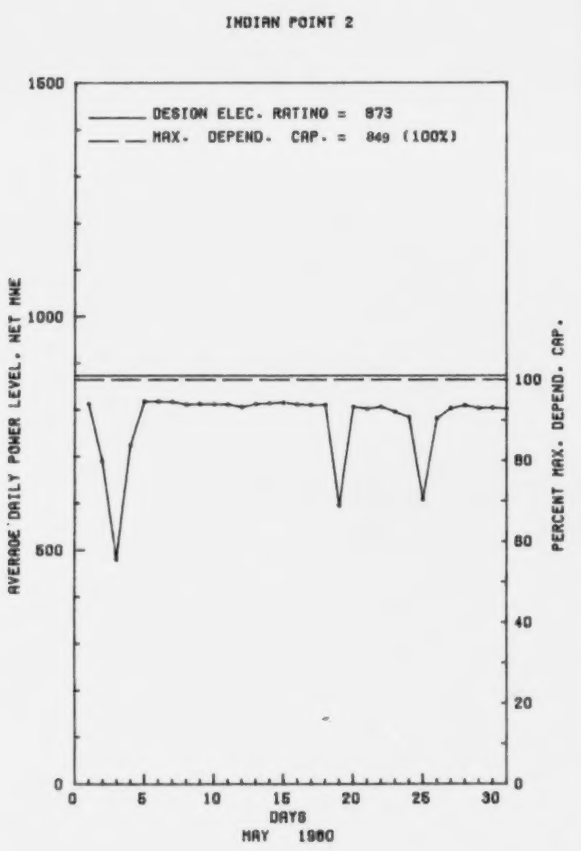
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-002/03L	04/09/80	05/09/80	Surveillance test of #22 diesel generator revealed that cylinder head temperatures were higher than normal due to a cylinder valve timing problem

REVIEWED BY OMPA LOEB: T. C. Cintoca **DATE:** 7·3·80

REVIEWED BY NRR DOR: **DATE:** 7-7-80

INDIAN POINT 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-247
2. Utility Contact: E. EICH (914) 694-6000 @ I.P.
3. Licensed Thermal Power (MWt): 2758
4. Nameplate Rating (Gross MWe): 1126 x 0.9 = 1013
5. Design Electrical Rating (Net MWe): 873
6. Maximum Dependable Capacity (Gross MWe): 885
7. Maximum Dependable Capacity (Net MWe): 849
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): 824
10. Reasons for Restrictions, If Any:
REMOVAL OF NO. 3 DISC. (GEN. END) ON NO 23 LOW PRESSURE

ITEM 21 CALCULATED BY USING A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	3,647.0	51,888.0
12. Hours Reactor Was Critical	742.0	2,783.8	35,078.3
13. Reactor Reserve Shutdown Hours	.0	852.4	1,093.8
14. Hours Generator On-Line	740.4	2,726.1	34,115.3
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,970,684	7,125,175	87,957,701
17. Gross Electrical Energy (MWH)	602,430	2,160,450	27,293,536
18. Net Electrical Energy (MWH)	577,944	2,059,545	26,024,645
19. Unit Service Factor	99.5	74.7	65.7
20. Unit Availability Factor	99.5	74.7	65.7
21. Unit Capacity Factor (MDC Net)	91.5	66.6	50.2
22. Unit Capacity Factor (DER Net)	89.0	64.7	57.5
23. Unit Forced Outage Rate	.5	6.9	8.0

24. Shutdowns Scheduled Over Next 6 Months (Type,Date,and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	05/03/80	S	.0	B	5		HF	PUMPXX	NO. 26 CIRCULATING WATER PUMP INSPECTION.
7	05/19/80	F	3.6	G	3		RB	CRDRVE	IMPROPER SEQUENCING OF BREAKERS WHILE RETURNING NO. 22 CRDM M/G SET TO SERVICE.
	05/25/80	F	.0	A	5		HH	PUMPXX	NO. 22 HDTP REPAIR.

SUMMARY

THE PLANT RAN AT REDUCED POWER LEVELS DURING MAY EXCEPT FOR A TRIP ON 05/19/80 FOLLOWING THE RETURN OF CRD MOTOR GEN. TO SERVICE. REDUCTIONS OCCURRED ON 05/03/80 AND ON 05/25/80 FOR PUMP REPAIR.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

INDIAN POINT 3

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	New York/Westchester 25 miles N of New York City
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	3025
5.	Design Electrical Rating (MWE Net)	965
6.	Date of Initial Criticality	April 6, 1976
7.	Date of Commercial Operation	August 30, 1976
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Hudson River

Utility & Contractor Information

10.	Licensee	Power Authority of the State of New York, Inc.
11.	Corporate Address	10 Columbus Circle New York, New York
12.	Corporate Contact	Peter N. Lyon - Senior Vice President
13.	Architect/Engineer	UE&C
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	WEDCO

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Section Chief	H. B. KISTER
19.	IE Resident Inspector	T. Rebelowski (914-739-9360) FTS 656-9220
20.	Licensing Project Manager	L. Olshan x27144
21.	Docket Number	50-286
22.	License Number/Date of Issue	DPR-64, 12/12/76
23.	Public Document Room	White Plains Public Library 100 Martine Avenue White Plains, New York 10601

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

MANAGERIAL ITEMS:

Mr. Bayne, the Resident Manager has been appointed Senior Vice President, Nuclear Generation. Mr. S. Zulla the Superintendent of Power has been appointed Acting Resident Manager effective May 29, 1980.

PLANT STATUS:

Routine operations.

LAST IE SITE INSPECTION DATE: 6/7/80

INSPECTION REPORT NO: 50-286/80-07

SUMMARY

INSPECTION SUMMARY

50-286/79-29 - Nov 25, 1979 - Jan 19, 1980: Routine, unannounced inspection by the resident inspectors (99 hrs) of plant operations: log and record review; tours of the facility; observation of physical security; radiation protection observation; pipe support and restraint systems review; emergency planning meetings; maintenance activities; refueling surveillance-preparation for plant startup; and seismic event review. Two items of noncompliance: (Failure to properly evaluate surveillance results; and Failure to evaluate and resolve discrepancies noted during a Containment Inspection).

50-286/80-04 - Mar 12-14: Routine, announced inspection by one regionally-based inspector (17 hrs) of the fire prevention/protection program pertaining to fire protection modifications outlined in Amendment 24 to License No. DPR-64 including: fire brigade training, manual fire fighting equipment, and administrative procedures. No items of noncompliance were identified.

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT

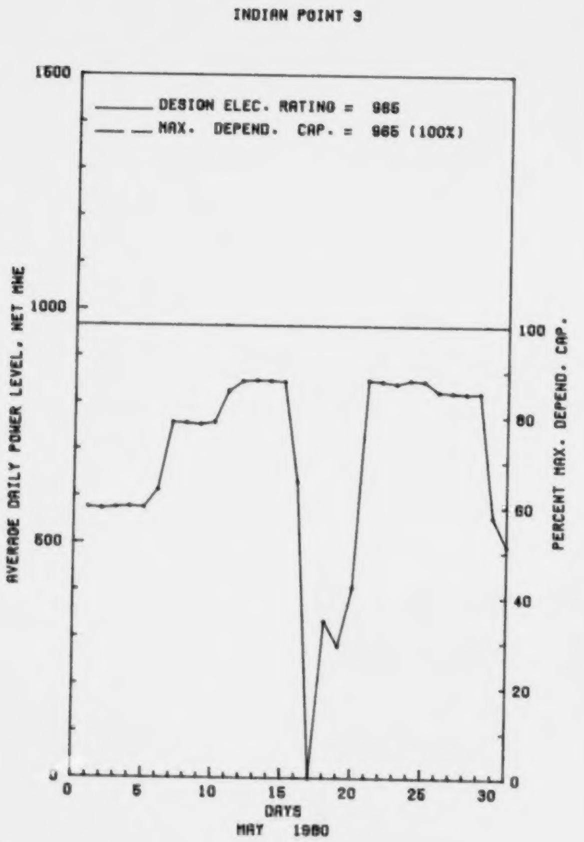
NONE

REVIEWED BY OMPA LOEB: T. C. Cintula **DATE:** 7·3·80

REVIEWED BY NRR DOR: **DATE:** 7-7-80

INDIAN POINT 3

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-286
2. Utility Contact: C. Connell (914) 739-8200 x242
3. Licensed Thermal Power (MWt): 3025
4. Nameplate Rating (Gross MWe): 1126 x 0.9 = 1013
5. Design Electrical Rating (Net MWe): 965
6. Maximum Dependable Capacity (Gross MWe): 1000
7. Maximum Dependable Capacity (Net MWe): 965
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): 817
10. Reasons for Restrictions, If Any: REMOVED BLADING FROM NO. 31 L.P. TURBINE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	3,647.0	32,904.0
12. Hours Reactor Was Critical	723.9	2,346.3	23,931.2
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	698.1	1,932.7	22,965.5
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,824,582	3,810,636	59,915,103
17. Gross Electrical Energy (MWH)	526,870	1,044,700	19,439,801
18. Net Electrical Energy (MWH)	503,021	981,862	18,625,299
19. Unit Service Factor	93.8	53.0	69.8
20. Unit Availability Factor	93.8	53.0	69.8
21. Unit Capacity Factor (MDC Net)	70.1	27.9	58.7
22. Unit Capacity Factor (DER Net)	70.1	27.9	58.7
23. Unit Forced Outage Rate	6.2	24.6	6.3

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): TURBINE OUTAGE-OCTOBER.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
16	05/16/80	F	15.5	A	3		EA	RELAYX	MISOPERATION OF 1R RELAY AT SUBSTATION IN BUCHANAN WITH MULTIPLE GROUND PATH BETWEEN IP3 AND SUBSTATION. GROUND ASSOCIATED WITH BATTERY 31 CLEARED UP.
17	05/19/80	F	22.7	A	3		ED	GENERA	VOLTAGE TRANSIENT ON 33 INSTRUMENT BUS REPAIRED COMPONENTS WITHIN 33 STATIC INVERTER.
18	05/30/80	F	7.7	A	3		HA	INSTRU	FALSE ACTUATION OF INDEPENDENT ELECTRIC TURBINE OVERSPEED PROTECTION SYSTEM.

SUMMARY

THE REACTOR BEGAN THE MONTH AT REDUCED POWER FOR CIRC WATER PUMP REPAIR. THE REACTOR REACHED FULL POWER ON 05/11/80 AND OPERATED AT THAT LEVEL, EXCEPT FOR THREE SCRAMS ON 05/16/80 BECAUSE OF LOSS OF LOAD, ON 05/19/80 DUE TO VOLTAGE TRANSIENT, AND ON 05/30/80 BECAUSE OF FALSE ACTUATION OF T.O.P.S.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

KEWAUNEE

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Wisconsin/Kewaunee 27 miles E of Green Bay
2.	Electric Reliability Council	Mid-America Interpool Network
3.	Type of Reactor	PWR
4.	Power Level (MWT)	1650
5.	Design Electrical Rating (MWE Net)	535
6.	Date of Initial Criticality	March 7, 1974
7.	Date of Commercial Operation	June 1974
8.	Condenser Cooling Method	Once through condenser and return to Lake Michigan
9.	Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10.	Licensee	Wisconsin Public Service Corp.
11.	Corporate Address	P. O. Box 1200 Green Bay, Wisconsin 54305
12.	Corporate Contact	E. R. Mathews Power Supply & Eng.
13.	Architect/Engineer	Fluor Power Services Corp.
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Pioneer Services & Eng. Co.

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	R. F. Warnick
19.	IE Resident Inspector	N. C. Choules
20.	Licensing Project Manager	R. B. Licciardo X28362
21.	Docket Number	50-305
22.	License Number/Date of Issue	DPR-43 issued 12/21/73
23.	Public Document Room	Kewaunee Public Library 822 Juneau Street Kewaunee, Wisconsin 54216

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

FACILITY ITEMS

PLANT STATUS

The plant is presently shutdown for refueling and is expected to return to power about June 20, 1980.

LAST IE SITE INSPECTION DATE: April 21-24, 1980

INSPECTION REPORT NO: 80-08

SUMMARY

INSPECTION SUMMARY

Inspection on March 24-28, (80-06): Routine, announced inspection of the licensee's Small Break LOCA emergency procedures, to determine compliance with the operator guidelines submitted by Westinghouse and as approved by the NRC Staff by letter dated November 5, 1979 to Mr. Cordell Reed, Chairman of Westinghouse Owner's Group from Mr. D. F. Ross, Jr., Director, Bulletins and Orders Task Force. The inspection also included the reviews of systems utilized in the event of a Small Break Loss of Coolant Accident. The inspection involved 35 inspector hours onsite by one NRC inspector. An NRC inspector in training was also onsite March 24-28, 1980. Of the three areas inspected, no items of noncompliance or deviations were found in two areas; one item of apparent deviation was identified in one area (failure to implement approved Westinghouse guidelines).

Inspection on April 21-25, (80-08): Routine, unannounced inspection of plant operations, fire protection/prevention program, nonroutine events, and IE Bulletin and Circular followup. The inspection involved 28 inspector hours onsite by one NRC inspector. The inspector was accompanied by two NRC inspectors in training. No items of noncompliance or deviations were identified.

REPORTS RECEIVED FROM LICENSEE

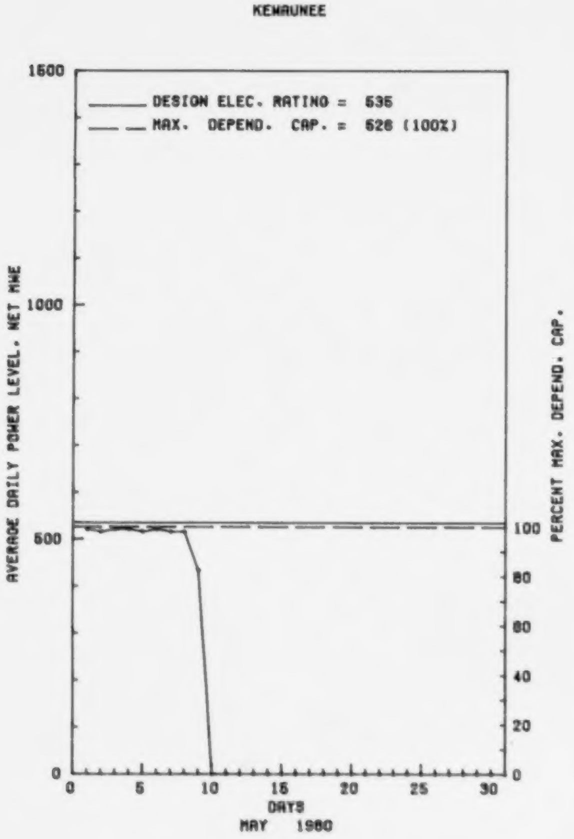
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-19/ 03L-0	03/26/80	04/23/80	Shield building ventilation system check damper failed to fully open.
80-20/ 03L-0	03/26/80	04/23/80	Control valve failed to open upon removal of heat detector.

REVIEWED BY OMPA LOEB: T. C. Cintula DATE: 7-3-80

REVIEWED BY NRR DOR: R Licciardo DATE: 7-7-80

KEWAUNEE

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-305
2. Utility Contact: G.RUITER (414) 388-2560 X207
3. Licensed Thermal Power (MWt): 1650
4. Nameplate Rating (Gross MWe): 622 x 0.9 = 560
5. Design Electrical Rating (Net MWe): 535
6. Maximum Dependable Capacity (Gross MWe): 545
7. Maximum Dependable Capacity (Net MWe): 526
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

ITEM 21 CALCULATED BY USING A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	3,647.0	52,248.0
12. Hours Reactor Was Critical	215.7	2,851.7	43,856.2
13. Reactor Reserve Shutdown Hours	.0	152.7	2,330.5
14. Hours Generator On-Line	215.7	2,836.5	42,830.5
15. Unit Reserve Shutdown Hours	.0	.0	10.0
16. Gross Thermal Energy (MWH)	349,621	4,586,111	65,737,234
17. Gross Electrical Energy (MWH)	115,400	1,524,300	21,708,100
18. Net Electrical Energy (MWH)	109,975	1,455,157	20,654,235
19. Unit Service Factor	29.0	77.7	82.0
20. Unit Availability Factor	29.0	77.7	82.0
21. Unit Capacity Factor (MDC Net)	28.1	75.9	78.4
22. Unit Capacity Factor (DER Net)	27.6	74.6	73.9
23. Unit Forced Outage Rate	.0	9.1	5.6

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 06/20/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
4	05/09/80	S	528.3	C	1		RC	FUELXX	COMMENCED CYCLE V-VI REFUELING OUTAGE.

SUMMARY

THE PLANT OPERATED AT NEAR FULL POWER UNTIL SHUTDOWN 05/08/80 FOR A REFUELING AND MAINTENANCE OUTAGE.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram.
- 3--Automatic Scram.
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

NRC POWER RESTRICTIONS: NONE

LACROSSE

FACILITY DATA

Facility Description

1. Location (state/county) (nearest population center)	Wisconsin/Vernon 19 miles S of LaCrosse
2. Electric Reliability Council	Mid-Continent Area Reliability Coordination Agreement
3. Type of Reactor	BWR
4. Power Level (MWT)	165
5. Design Electrical Rating (MWE Net)	50
6. Date of Initial Criticality	July 11, 1967
7. Date of Commercial Operation	November 1, 1969
8. Condenser Cooling Method	Once Through
9. Condenser Cooling Water Source	Mississippi River

Utility & Contractor Information

10. Licensee	Dairyland Power Cooperative
11. Corporate Address	2615 East Avenue South La Crosse, Wisconsin
12. Corporate Contact	F. Linder
13. Architect/Engineer	Sargent & Lundy
14. Nuclear Steam Supply System	Allis-Chalmers
15. Constructor	Maxson Construction

Regulatory Information

16. IE Region responsible	III
17. IE Branch Chief	R. F. HEISHMAN
18. IE Section Chief	K. R. BAKER
19. IE Resident Inspector	K. Ridgway
20. Licensing Project Manager	J. J. Shea
21. Docket Number	50-409
22. License Number/Date of Issue	DPR-45 issued 8/28/73
23. Public Document Room	LaCrosse Public Library 800 Main Street LaCrosse, Wisconsin 54601

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

Plant is operating routinely.

LAST IE SITE INSPECTION DATE: April 28-29, 1980

INSPECTION REPORT NO: 80-03

INSPECTION SUMMARY

Inspection on April 28-29, (80-03): Spent fuel rack QA/QC programs, implementing welding and nondestructive examinations (NDE) procedures and record review. The inspection involved nine inspector hours onsite by one NRC inspector. No items of noncompliance or deviations were identified.

REPORTS RECEIVED FROM LICENSEE

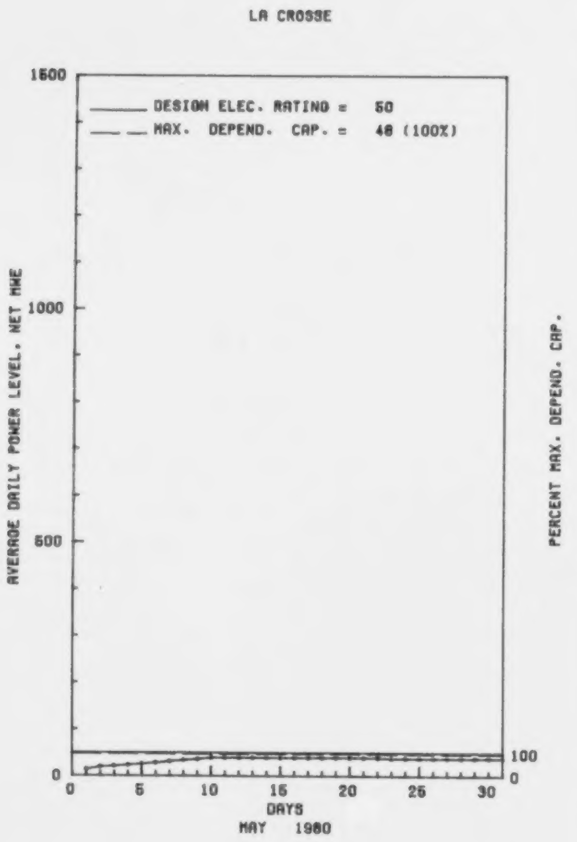
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-03/ 01T-0	04/17/80	04/29/80	Failure of steam condensate return valve to meet Type C leak rate test.
80-04/ 03L-0	04/17/80	05/13/80	During TS testing (TS 5.2.6) while shutdown, the 1A reactor emergency flooding vent valve did not open.

REVIEWED BY OMPA LOEB: John J. Ruzzo DATE: 7-2-80

REVIEWED BY NRR DOR: James J Shea DATE: 7-8-80

LACROSSE

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-409
2. Utility Contact: L. S. Goodman (608) 689-2331
3. Licensed Thermal Power (MWt): 165
4. Nameplate Rating (Gross MWe): 76.8 x 0.85 =65
5. Design Electrical Rating (Net MWe): 50
6. Maximum Dependable Capacity (Gross MWe): 50
7. Maximum Dependable Capacity (Net MWe): 48
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	3,647.0	92,762.0
12. Hours Reactor Was Critical	744.0	3,082.4	60,888.5
13. Reactor Reserve Shutdown Hours	.0	.0	478.0
14. Hours Generator On-Line	744.0	2,990.9	55,995.9
15. Unit Reserve Shutdown Hours	.0	.0	79.0
16. Gross Thermal Energy (MWH)	95,511	404,395	7,689,174
17. Gross Electrical Energy (MWH)	27,710	118,730	2,317,862
18. Net Electrical Energy (MWH)	25,939	111,230	2,143,965
19. Unit Service Factor	100.0	82.0	60.4
20. Unit Availability Factor	100.0	82.0	60.5
21. Unit Capacity Factor (MDC Net)	72.6	63.5	48.2
22. Unit Capacity Factor (DER Net)	69.7	61.0	46.2
23. Unit Forced Outage Rate	.0	2.4	6.2

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): OPERATOR LICENSING EXAMINATIONS, JUNE 2-4, 1980 ; REFUELING, 09/20/80
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	NONE								

SUMMARY

OPERATION AT 85% POWER TO EXTEND CORE LIFE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

MAINE YANKEE

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Maine/Lincoln 10 miles N of Bath (near Wiscasset)
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2630
5.	Design Electrical Rating (MWE Net)	825
6.	Date of Initial Criticality	October 23, 1972
7.	Date of Commercial Operation	December 28, 1972
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Back River (Atlantic)

Utility & Contractor Information

10.	Licensee	Maine Yankee Atomic Power Co.
11.	Corporate Address	20 Turnpike Road Westboro, Massachusetts 01581
12.	Corporate Contact	D. W. Edwards, Assistant to Vice President
13.	Architect/Engineer	Stone & Webster
14.	Nuclear Steam Supply System	Combustion Engineering
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	
17.	IE Branch Chief	E. Brunner
18.	IE Section Chief	T. T. Martin
19.	IE Resident Inspector	W. J. LAZARUS (X5258)
20.	Licensing Project Manager	G. Requa (X28478)
21.	Docket Number	50-309
22.	License Number/Date of Issue	DPR-36, 9/15/72
23.	Public Document Room	Wiscasset Public Library High Street Wiscasset, Maine 04578

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

Two spurious reactor trips occurred because of noise spikes in the reactor protection system. Concern over increasing core pressure drop has resulted in two attempts at chemical cleaning of the core to reduce crud buildup on fuel pins. One steam generator non-return valve is not operating. The plant is now in cold shutdown for repair of this valve.

PLANT STATUS:

Shutdown on 5/30/80 was to repair the No. one non-return valve, replace the No. 2 reactor coolant pump seal, trouble-shoot the reactor protection system, and perform chemical cleaning of the core.

LAST IE SITE INSPECTION DATE: 5/30/80

INSPECTION REPORT NO: 50-309/80-08

SUMMARY

INSPECTION SUMMARY

None

REPORTS RECEIVED FROM LICENSEE

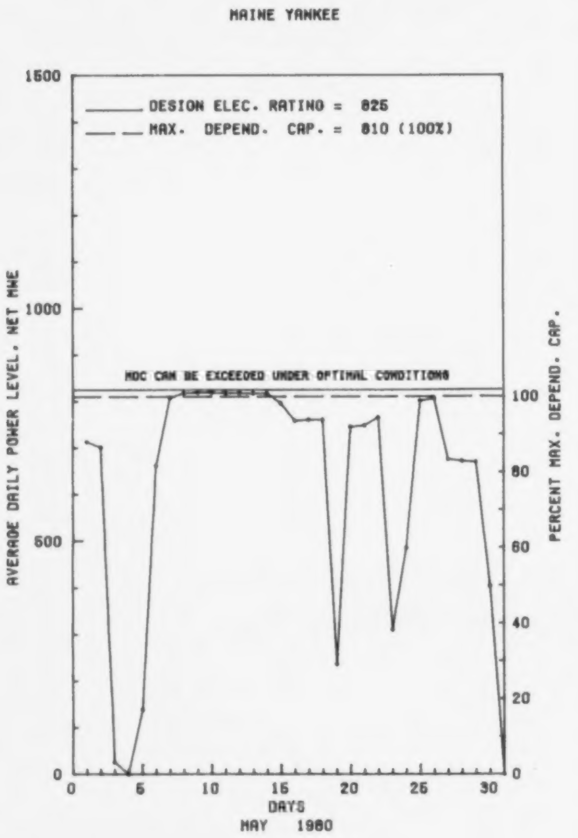
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-010/ 03L	04/08/80	05/06/80	Auxiliary feedwater pump inoperable

REVIEWED BY OMPA LOEB: P E Bobe **DATE:** 7/3/80

REVIEWED BY NRR DOR: G. Requa **DATE:** 7/7/80

MAINE YANKEE

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-309
2. Utility Contact: F.C. BEERS (617) 366-9011
3. Licensed Thermal Power (MWt): 2630
4. Nameplate Rating (Gross MWe): 900 x 0.9 = 810
5. Design Electrical Rating (Net MWe): 825
6. Maximum Dependable Capacity (Gross MWe): 850
7. Maximum Dependable Capacity (Net MWe): 810

CAPACITY FACTORS CALCULATED WITH A WEIGHTED AVERAGE

8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	3,647.0	66,275.6
12. Hours Reactor Was Critical	640.0	2,120.5	53,330.4
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	631.0	1,981.1	51,498.8
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,449,738	4,187,013	109,429,096
17. Gross Electrical Energy (MWH)	483,080	1,384,450	35,957,630
18. Net Electrical Energy (MWH)	459,925	1,315,200	34,142,386
19. Unit Service Factor	84.8	54.3	77.7
20. Unit Availability Factor	84.8	54.3	77.7
21. Unit Capacity Factor (MDC Net)	76.3	44.5	66.8
22. Unit Capacity Factor (DER Net)	74.9	43.7	62.4
23. Unit Forced Outage Rate	3.7	4.5	6.8

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 06/13/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	04/24/80	S	51.0	H	5		RC	ZZZZZZ	LOAD REDUCTION DUE TO INCREASING CORE D/P. NH4OH WAS ADDED TO HELP REMOVE THE APPARENT BUILDUP OF CRUD ON CORE SURFACES. (75% POWER)
5-80-5	05/03/80	S	57.6	H	1		RC	ZZZZZZ	PLANT SHUTDOWN AND COOLDOWN FOR CORE CRUD CLEANUP. HYDROGEN PEROXIDE WAS ADDED TO ATTEMPT TO REMOVE THE APPARENT BUILDUP OF CRUD ON CORE SURFACES.
	05/15/80	S	84.9	H	5		RC	ZZZZZZ	LOAD REDUCTION TO REDUCE HEAT FLUX IN CONSIDERATION TO INCREASING CORE D/P. (90% POWER)
6-80-5	05/19/80	F	9.3	A	3		IA	INSTRU	SPURIOUS ELECTRICAL SPIKE RESULTED IN TM/LP TRIP ON RPS CHANNELS C & D. INVESTIGATION DID NOT REVEAL CAUSE OF SPIKES.
7-80-5	05/24/80	F	14.9	A	3		IA	INSTRU	SPURIOUS ELECTRICAL SPIKE RESULTS IN TM/LP TRIP ON RPS CHANNELS C & D. INVESTIGATION INDICATES POSSIBLE INTERACTION FROM THE ANNUNCIATOR TEST CIRCUITRY.
	05/27/80	F	88.3	H	5		RC	ZZZZZZ	LOAD REDUCTION TO REDUCE HEAT FLUX IN CONSIDERATION TO INCREASED CORE D/P. (80% POWER)
8-80-5	05/30/80	S	31.2	H	1		RC	ZZZZZZ	SHUTDOWN TO ACCOMPLISH CHEMICAL CLEAN OF THE RCS TO REMOVE CORE CRUD.

SUMMARY

THE PLANT BEGAN THE MONTH AT 85% POWER AND WAS SHUTDOWN ON 05/03/80 AND 05/30/80 FOR CORE CRUD CLEANING. THE UNIT OPERATED AT FULL POWER UNTIL 05/27/80 WHEN POWER WAS REDUCED AGAIN BECAUSE OF CRUD BUILDUP. SCRAMS OCCURRED ON 05/19/80 AND ON 05/24/80 DUE TO FALSE TEMPERATURE SIGNALS.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram.
3--Automatic Scram.
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
ame source.

NRC POWER RESTRICTIONS: NONE

MILLSTONE 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Connecticut/New London 5 miles SW of New London
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2011
5.	Design Electrical Rating (MWE Net)	660
6	Date of Initial Criticality	October 26, 1970
7.	Date of Commercial Operation	March 1971
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Long Island Sound

Utility & Contractor Information

10.	Licensee	Northeast Nuclear Energy Company
11.	Corporate Address	P. O. Box 270 Hartford, Connecticut
12.	Corporate Contact	W. G. Counsil, Vice President
13.	Architect/Engineer	Ebasco
14.	Nuclear Steam Supply System	GE
15.	Constructor	Ebasco

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	IE Section Chief	R. KEIMIG
19.	IE Resident Inspector	J. SHEDLOSKY (203-447-3179) FTS: 642-7287
20.	Licensing Project Manager	J. J. Shea
21.	Docket Number	50-245
22.	License Number/Date of Issue	DPR-21, 10/7/70
23.	Public Document Room	Waterford Public Library Rope Ferry Road Route 156 Waterford, Connecticut 06385

INSPECTION STATUS

ENFORCEMENT STATUS

10 CFR 50, APP B, CRITERION XVI, STATES IN PART: "MEASURES SHALL BE ESTABLISHED TO ASSURE THAT CONDITIONS ADVERSE TO QUALITY, SUCH AS FAILURES, MALFUNCTIONS, DEFICIENCIES, DEVIATIONS, DEFECTIVE MATERIAL & EQUIPMENT, & NONCONFORMANCES ARE PROMPTLY IDENTIFIED & CORRECTED. IN CASE OF SIGNIFICANT CONDITIONS ADVERSE TO QUALITY...THE CAUSE OF THE CONDITION, & THE CORRECTIVE ACTION TAKEN SHALL BE DOCUMENTED & REPORTED TO APPROPRIATE LEVELS OF MANAGEMENT. CONTRARY TO THE ABOVE, AS OF 1/3/80, MEASURES HAD NOT BEEN ESTABLISHED TO ASSURE THAT CONDITIONS ADVERSE TO QUALITY WERE ADEQUATELY IDENTIFIED, DOCUMENTED, EVALUATED & CORRECTED. THE FOLLOWING ARE EXAMPLES: THE CRACKS IN CONCRETE OF THE DRYWELL WALL IN THE VICINITY OF PENETRATION X-10A WERE NOT PROPERLY & ADEQUATELY EVALUATED; &, THE REPAIRS PERFORMED ON THE LOOSENED ANCHOR BOLTS IN PIPE SUPPORTS OF THE ISOLATION CONDENSER SYSTEM WERE NOT PROPERLY DOCUMENTED.
(8002 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS:

The plant is operating at 84% power and 515 MW(e) on the advice of the turbine manufacturer, due to a failed expansion joint on a turbine extraction steam line. A one week outage, for repairs, is planned for 5/30. Stack and offgas release rates are 330 microcuries per second and 140,000 microcuries per second, respectively.

LAST IE SITE INSPECTION DATE: 5/31/80

INSPECTION REPORT NO: 50-245/80-07

SUMMARY

INSPECTION SUMMARY

50-245/80-03 - Feb 1-29: Routine, onsite regular and backshift inspection by two resident inspectors (122 hours). Areas inspected included the control rooms and the accessible portions of the reactor, turbine, radioactive waste, gas turbine generator, and intake buildings; the condensate polishing facility; radiation protection; physical security; fire protection; plant operating records; surveillance testing; calibration; maintenance; core power distribution limits; and reporting to the NRC. No items of noncompliance were identified.

50-245/80-05 - Mar 13: A meeting was convened to discuss the findings of an investigation conducted by Region I of several allegations pertaining to the water-hammer event of December 19, 1979, and subsequent followup activities by the licensee and NRC. Licensee actions involved in system modifications and repair were also discussed.

REPORTS RECEIVED FROM LICENSEE

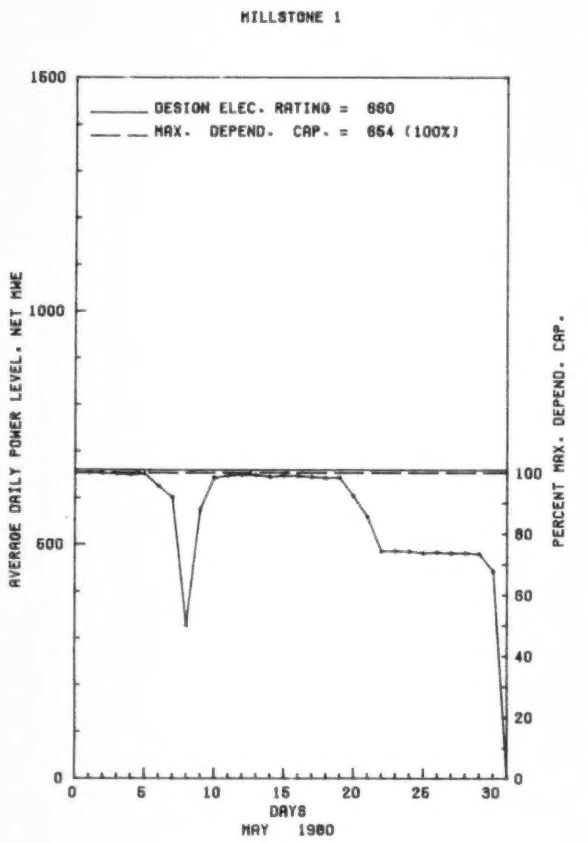
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-006/03L	04/07/80	04/21/80	One of four drywell pressure switches found isolated during routine plant tour

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 7-3-80

REVIEWED BY NRR DOR: James J. Shea **DATE:** 7-8-80

MILLSTONE 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-245
2. Utility Contact: R. Young (203) 447-1792 x475
3. Licensed Thermal Power (MWt): 2011
4. Nameplate Rating (Gross MWe): 735 x 0.9 = 662
5. Design Electrical Rating (Net MWe): 660
6. Maximum Dependable Capacity (Gross MWe): 684
7. Maximum Dependable Capacity (Net MWe): 654
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	3,647.0	83,327.0
12. Hours Reactor Was Critical	724.5	3,627.5	64,012.5
13. Reactor Reserve Shutdown Hours	19.5	19.5	954.5
14. Hours Generator On-Line	720.5	3,623.5	61,719.4
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,272,306	5,949,889	109,997,834
17. Gross Electrical Energy (MWH)	435,100	2,018,400	37,249,796
18. Net Electrical Energy (MWH)	415,028	1,926,465	35,566,247
19. Unit Service Factor	96.8	99.4	74.1
20. Unit Availability Factor	96.8	99.4	74.1
21. Unit Capacity Factor (MDC Net)	85.3	80.8	65.3
22. Unit Capacity Factor (DER Net)	84.5	80.0	64.7
23. Unit Forced Outage Rate	.0	.0	15.8

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): ANNUAL REFUEL OUTAGE SCHED TO COMMENCE 09/19/80 FOR 8-10 WKS
25. If Shut Down At End of Report Period, Estimated Date of Startup: 06/22/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	05/08/80	S	15.0	B	5		HC	HTEXCH	REPAIR CONDENSER TUBE LEAKS AND ROD PATTERN ADJUSTMENTS.
03	05/30/80	S	23.5	A	9		HJ	TURBIN	UNIT WAS REMOVED FROM SERVICE TO INSPECT AND REPAIR AS NECESSARY A STEAM LEAK IN THE EXTRACTION STEAM EXPANSION JOINT OFF L.P. TURBINE.

SUMMARY

THE UNIT OPERATED AT 100% DURING THE PERIOD EXCEPT FOR REDUCTIONS ON 5/6, 5/8, AND 5/10 FOR CONDENSER PROBLEMS AND ON 05/14/80 FOR TURBINE STOP VALVE TESTING. THE REACTOR BEGAN TO LOSE POWER ON 05/20/80 BECAUSE OF SUSPECTED TURBINE LEAKS AND WAS SHUTDOWN ON 05/30/80 FOR LEAK REPAIR.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

MILLSTONE 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Connecticut/New London 5 miles SW of New London
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2700
5.	Design Electrical Rating (MWE Net)	870
6.	Date of Initial Criticality	October 17, 1975
7.	Date of Commercial Operation	December 26, 1975
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Long Island Sound

Utility & Contractor Information

10.	Licensee	Northeast Nuclear Energy Company
11.	Corporate Address	P. O. Box 270 Hartford, Connecticut
12.	Corporate Contact	W. G. Counsil, Vice President
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Combustion Engineering
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	IE Section Chief	E. McCabe
19.	IE Resident Inspector	J. T. SHEDLOSKY (203-447-3179) FTS: 642-7287 R. Zimmerman
20.	Licensing Project Manager	E. L. Conner, Jr.
21.	Docket Number	50-336
22.	License Number/Date of Issue	DPR-65, 8/1/75
23.	Public Document Room	Waterford Public Library Rope Ferry Road Route 156 Waterford, Connecticut 06385

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS:

The plant has been shutdown since 5/8/80, when it was discovered that several safety related pipe supports, in the service water system, were not designed to a design basis earthquake. The unit remains shutdown while the licensee's architect engineer verifies the factor of safety of 164 additional safety related pipe supports.

LAST IE SITE INSPECTION DATE: 5/31/80

INSPECTION REPORT NO: 50-336/80-05

SUMMARY

INSPECTION SUMMARY

50-336/80-02 - Feb 1-29: Routine, onsite regular and backshift inspection by two resident inspectors (84 hours). Areas inspected included the control rooms, and the accessible portions of the enclosure, auxiliary, turbine and intake buildings; the condensate polishing facility; radiation protection; physical security; fire protection; plant operating records; surveillance testing; calibration; maintenance; core power distribution limits; and reporting to the NRC. No items of noncompliance were identified.

REPORTS RECEIVED FROM LICENSEE

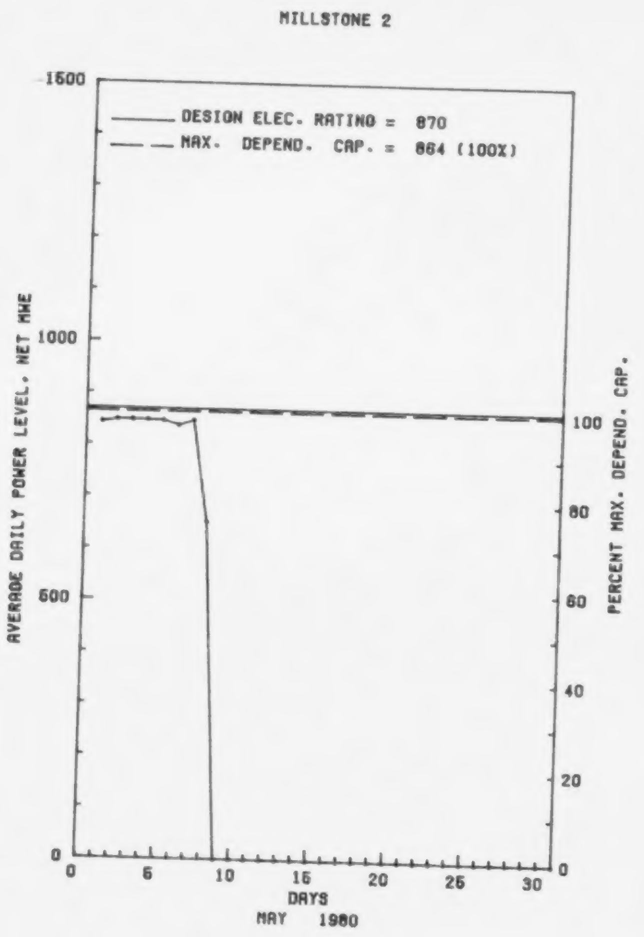
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-013/ 03L	03/20/80	04/18/80	CEA dropped during movement. Operation continued per TS. No limits were exceeded.
80-014/ 03L	03/27/80	04/28/80	"A" control room exhaust fan placed out of service to replace noisy motor bearings
80-015/ 03L	03/21/80	04/18/80	Condensate storage tank level below TS limit. Plant operated in accordance with TS
80-016/ 03L	04/03/80	05/01/80	Setpoint drift in RPS Channel A low steam generator pressure bypass
80-017/ 04T	04/24/80	05/05/80	Inoperable discharge flow recorder
80-019/ 03L	04/11/80	05/13/80	Reactor coolant pressure decreased to less than TS limit during pressure adjustment with the spray valve

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 7-3-80

REVIEWED BY NRR DOR: Elan L. Conner, Jr. **DATE:** 7/7/80

MILLSTONE 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-336
2. Utility Contact: G. H. Howlett (203) 447-1791 x364
3. Licensed Thermal Power (MWt): 2700
4. Nameplate Rating (Gross MWe): 1011 x 0.9 = 910
5. Design Electrical Rating (Net MWe): 870
6. Maximum Dependable Capacity (Gross MWe): 895
7. Maximum Dependable Capacity (Net MWe): 864

CAPACITY FACTORS CALCULATED WITH A WEIGHTED AVERAGE

8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	3,647.0	38,855.0
12. Hours Reactor Was Critical	191.5	3,004.6	28,083.3
13. Reactor Reserve Shutdown Hours	.0	.0	2,072.4
14. Hours Generator On-Line	189.5	2,933.7	26,753.9
15. Unit Reserve Shutdown Hours	.0	.0	335.4
16. Gross Thermal Energy (MWH)	499,545	7,757,966	65,455,674
17. Gross Electrical Energy (MWH)	163,563	2,537,378	21,122,023
18. Net Electrical Energy (MWH)	154,711	2,439,127	20,219,385
19. Unit Service Factor	25.5	80.4	68.9
20. Unit Availability Factor	25.5	80.4	69.7
21. Unit Capacity Factor (MDC Net)	24.1	77.4	63.4
22. Unit Capacity Factor (DER Net)	23.9	76.9	62.1
23. Unit Forced Outage Rate	74.5	19.6	23.4

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
REFUELING (CYCLE 4), JULY 26, 1980, 9 WEEKS
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWN/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
9	05/08/80	F	554.4	D	1				PIPE SUPPORT REEVALUATION AND MODIFICATIONS PER I & E BULLETIN 79-02.

SUMMARY

THE UNIT OPERATED THE FIRST EIGHT DAYS OF THE MONTH AT NEAR FULL POWER. THE REACTOR WAS THEN SHUTDOWN TO UPGRADE PIPE SUPPORTS.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

MONTICELLO

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Minnesota/Wright 30 miles NW of Minneapolis
2.	Electric Reliability Council	Mid-Continent Area Reliability Coordination Agreement
3.	Type of Reactor	BWR
4.	Power Level (MWT)	1670
5.	Design Electrical Rating (MWE Net)	545
6.	Date of Initial Criticality	December 10, 1970
7.	Date of Commercial Operation	June 30, 1971
8.	Condenser Cooling Method	Once Through with Cooling Tower
9.	Condenser Cooling Water Source	Mississippi River

Utility & Contractor Information

10.	Licensee	Northern States Power Co.
11.	Corporate Address	414 Nicollet Mall - 8th Floor Minneapolis, Minnesota 55401
12.	Corporate Contact	G. H. Neils (Oper.) L. O. Mayer (Lic.)
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	GE
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. F. HEISHMAN
18.	IE Section Chief	R. F. Warnick
19.	IE Resident Inspector	C. Brown
20.	Licensing Project Manager	T. Kevern
21.	Docket Number	50-263
22.	License Number/Date of Issue	DPR-22 issued 9/8/70
23.	Public Document Room	Environmental Conservation Library Minneapolis Public Library 300 Nicollet Mall Minneapolis, Minnesota 55401

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

MANAGERIAL ITEMS

Current plant manager is being promoted to a position in the corporate office effective April 1, 1980, but will stay at the plant until a new plant manager is selected and on board. (80-04)

PLANT STATUS

The plant is currently operating at 100%.

LAST IE SITE INSPECTION DATE: March 3, 1980

INSPECTION REPORT NO: 80-08

SUMMARY

INSPECTION SUMMARY

Inspection on February 21-22 and March 27-28, (80-04): Routine, announced inspection of operational radiation protection program during refueling and maintenance, including: organization and staffing; training, radiation protection procedures; planning, preparation, and ALARA; exposure control; posting and control; material control; surveys; spent resin disposal; and pipe fitters respiratory problem. The inspection involved 22.5 inspector hours onsite by one NRC inspector. No items of noncompliance or deviations were identified.

Inspection on March 19-21 and April 7-11, (80-07): Special, announced inspection of small break LOCA procedures and routine unannounced inspection of relief valve position indication, LER followup, IEB and IEC followup, plant operations, onsite review committee, fire protection and prevention, and calibration. The inspection involved 118 inspector hours onsite by 4 NRC inspectors. No items of noncompliance or deviations were identified.

Inspection on March 3, (80-08): During the inspection of radioactive waste shipments conducted at Richland, Washington from March 3-14, 1980, Trailer 03 (Freight Bill 80521) was inspected. A representative of the State of Washington also inspected the shipment. One apparent item of noncompliance with DOT regulations was identified (failure to mark a cask used to transport radioactive waste).

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-18/ 03L-0	04/11/80	05/09/80	Moisture separator level switch fitting crack.
80-19/ 03L-0	04/14/80	05/16/80	RHR service water pump No. 12 problem.

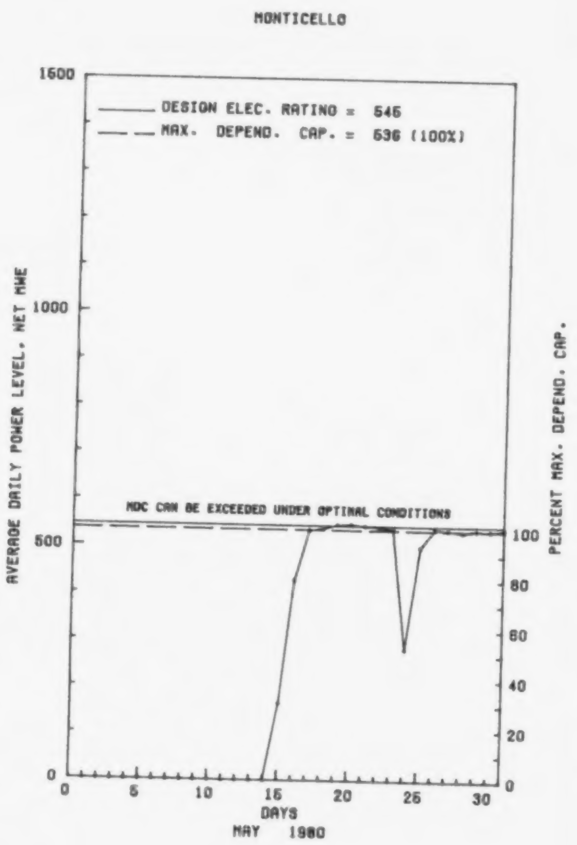
REVIEWED BY OMPA LOEB: John J. Ruzzo **DATE:** 7-3-80

REVIEWED BY NRR DOR: **DATE:** 8/7/80

GPO 871-867

MONTICELLO

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-263
2. Utility Contact: A. L. Myrabo (612) 295-5151
3. Licensed Thermal Power (MWt): 1670
4. Nameplate Rating (Gross MWe): 632 x 0.9 = 569
5. Design Electrical Rating (Net MWe): 545
6. Maximum Dependable Capacity (Gross MWe): 564
7. Maximum Dependable Capacity (Net MWe): 536
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	3,647.0	78,192.0
12. Hours Reactor Was Critical	410.8	2,050.9	63,591.5
13. Reactor Reserve Shutdown Hours	.0	.0	940.7
14. Hours Generator On-Line	398.2	1,970.9	61,941.0
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	624,055	2,714,184	101,657,046
17. Gross Electrical Energy (MWH)	211,060	914,610	31,892,670
18. Net Electrical Energy (MWH)	200,464	869,204	30,501,408
19. Unit Service Factor	53.5	54.0	79.2
20. Unit Availability Factor	53.5	54.0	79.2
21. Unit Capacity Factor (MDC Net)	50.3	44.5	72.8
22. Unit Capacity Factor (DER Net)	49.4	43.7	71.6
23. Unit Forced Outage Rate	46.5	21.8	7.1

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): MARCH 28, 1981 - REFUELING OUTAGE - 56 DAYS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type¹	Duration (Hours)	Reason²	Method of Shutting Down Reactor³	Licensee Event Report No.	System Code⁴	Component Code⁵	Cause & Corrective Action to Prevent Recurrence
8	04/29/80	F	345.8	A	4		CB	PUMPXX	CONTINUATION OF 04/29/80 OUTAGE TO REPAIR RECIRC PUMP SEALS.
9	05/24/80	S	.0	H	5		MC	HTEXCH	REDUCED POWER TO 30% TO REPAIR MAIN CONDENSER TUBE LEAK.

SUMMARY

THE PLANT RETURNED TO SERVICE 05/14/80 AFTER RECIRC PUMP SEAL REPAIR. THE UNIT OPERATED NEAR FULL POWER THE REST OF THE MONTH EXCEPT FOR A 30% REDUCTION FOR CONDENSER TUBE REPAIR.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

NINE MILE POINT 1

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	New York/Oswego 8 miles NE of Oswego
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	1850
5.	Design Electrical Rating (MWE Net)	620
6.	Date of Initial Criticality	September 5, 1969
7.	Date of Commercial Operation	December 1969
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Lake Ontario

Utility & Contractor Information

10.	Licensee	Niagara Mohawk Power Corporation
11.	Corporate Address	300 Erie Boulevard West Syracuse, New York 13202
12.	Corporate Contact	R. Schneider, Vice President Electric Operations
13.	Architect/Engineer	Utility
14.	Nuclear Steam Supply System	GE
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Section Chief	H. B. KISTER
19.	IE Resident Inspector	W. BAUNACK (X5253)
20.	Licensing Project Manager	P. Polk
21.	Docket Number	50-220
22.	License Number/Date of Issue	DPR-63, 12/26/74
23.	Public Document Room	State University College of Oswego Penfield Library - Documents Oswego, NY 13126 (315)341-2323

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS:

The plant is operating at essentially full power with representative offgas and stack release rates of 19,000 and 10 microcuries per second, respectively.

LAST IE SITE INSPECTION DATE: 5/23/80

INSPECTION REPORT NO: 50-220/80-05

SUMMARY

INSPECTION SUMMARY

None.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT

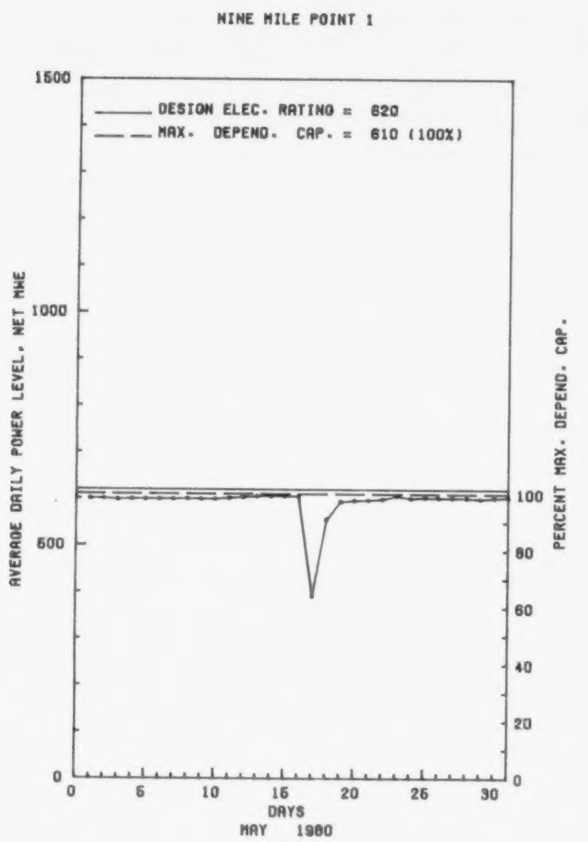
NONE

REVIEWED BY OMPA LOEB: John J. Rizzo DATE: 7-3-80

REVIEWED BY NRR DOR: Philip J. Polk DATE: 7/1/80

NINE MILE POINT 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-220
2. Utility Contact: B. Hawksley (315) 343-2110 x1312
3. Licensed Thermal Power (MWt): 1850
4. Nameplate Rating (Gross MWe): 755 x 0.85 = 642
5. Design Electrical Rating (Net MWe): 620
6. Maximum Dependable Capacity (Gross MWe): 630
7. Maximum Dependable Capacity (Net MWe): 610
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	3,647.0	92,759.0
12. Hours Reactor Was Critical	744.0	3,468.4	68,821.1
13. Reactor Reserve Shutdown Hours	.0	.0	1,204.2
14. Hours Generator On-Line	744.0	3,438.3	66,248.8
15. Unit Reserve Shutdown Hours	.0	.0	20.2
16. Gross Thermal Energy (MWH)	1,350,026	5,986,753	107,728,152
17. Gross Electrical Energy (MWH)	454,926	2,015,915	35,525,425
18. Net Electrical Energy (MWH)	441,011	1,950,983	34,400,748
19. Unit Service Factor	100.0	94.3	71.4
20. Unit Availability Factor	100.0	94.3	71.4
21. Unit Capacity Factor (MDC Net)	97.2	87.7	60.8
22. Unit Capacity Factor (DER Net)	95.6	86.3	59.8
23. Unit Forced Outage Rate	.0	.0	8.9

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-14	05/17/80	S	.0	B	5				LOAD REDUCTION TO 40% POWER TO INSPECT CONDENSER TUBES AND PLUG LEAKS.

SUMMARY

UNIT RAN AT FULL POWER FOR REPORTING MONTH WITH A 40% LOAD REDUCTION ON 05/17/80 DUE TO INSPECTION OF CONDENSER TUBES AND PLUG LEAKS.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

NORTH ANNA 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Louisa County, Virginia 40 Miles NW of Richmond, Virginia
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	PWR
4.	Licenses Power Level (MWT)	2775
5.	Design Electrical Rating (MWE Net)	907
6.	Date of Initial Criticality	April 5, 1978
7.	Date of Commercial Operation	June 6, 1978
8.	Condenser Cooling Method	Once through
9.	Condenser Cooling Water Source	Lake Anne

Utility & Contractor Information

10.	Licensee	Virginia Electric & Power Co.
11.	Corporate Address	P. O. Box 26666 Richmond, Virginia 23261
12.	Corporate Contact	J. H. Ferguson Exec. V.P. - Power
13.	Architect/Engineer	Stone & Webster
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. Lewis
18.	IE Section Chief	P. J. Kellog
19.	IE Resident Inspector	M. S. Kidd (703) 894-6927
20.	Licensing Project Manager	L. Engle
21.	Docket Number	50-338
22.	License Number/Date of Issue	NPF-4, 11/26/77
23.	Public Document Room	Alderman Library/Manuscripts Dept. Univ. of Virginia/Charlottesville VA 22901 & Louisa County Courthouse, Louisa, VA 23093

INSPECTION STATUS

ENFORCEMENT STATUS

AS REQUIRED BY 10 CFR 50 APPENDIX B CRITERIA XI AS IMPLEMENTED BY THE TOPICAL QUALITY ASSURANCE REPORT VEP-3-A SECTION 17.2.11 WRITTEN TEST PROCEDURES SHALL INCORPORATE THE REQUIREMENTS CONTAINED IN APPLICABLE SECTION 11, PARAGRAPH 2.1 FURTHER REQUIRES THAT TESTS DURING THE PREOPERATIONAL PERIOD SHALL DEMONSTRATE PERFORMANCE IN ACCORDANCE WITH DESIGN INTENT AS DESCRIBED IN THE FSAR. FSAR SECTION 7.3.1.3.5(I) AND FSAR COMMENT 7.4 BOTH IDENTIFY THE DESIGN FEATURE WHICH REQUIRES TWO SEPARATE OPERATOR ACTIONS TO RESET ANY ENGINEERED SAFEGUARDS FEATURE (ESF) ACTUATION CAUSING EQUIPMENT TO REVERT TO ITS NORMAL MODE OF OPERATION. CONTRARY TO THE ABOVE, THE NORTH ANNA UNIT 1 PREOPERATIONAL TEST PROGRAM HAD NO PROVISION TO TEST THE ESF RESET DESIGN TO ASSURE ITS OPERATION AS DESCRIBED IN FSAR SECTION 7.3.1.3.5(I) AND FSAR COMMENT 7.4.
(8020 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

UNIT WAS SHUTDOWN AT THE END OF MAY.

LAST IE SITE INSPECTION DATE: MAY 21, 1980

INSPECTION REPORT NO: 50-338/80-26

SUMMARY

INSPECTION SUMMARY

INSPECTION APRIL 9-10 (80-17): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 14 INSPECTOR-HOURS ON SITE IN THE AREAS OF INSPECTION ACTIVITIES RELATIVE TO IE BULLETIN 79-17, "PIPE CRACKS IN STAGNANT BORATED WATER SYSTEMS AT PWR PLANTS"; AND IE BULLETIN 80-08, "EXAMINATION OF CONTAINMENT LINER PENETRATION WELDS." NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION APRIL 13-16 (80-18): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 43 INSPECTOR-HOURS ON SITE. THE INSPECTION FOCUSED ON THE INITIAL FUEL LOADING OF UNIT 2. LICENSEE ACTIONS ON AN EARLIER INSPECTION FINDING ON UNIT 1 AND INSPECTOR-IDENTIFIED ISSUES WERE ALSO ADDRESSED. NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION APRIL 21-25 (80-20): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 11 HOURS ON SITE IN THE AREAS OF PLANT OPERATION, LICENSEE EVENT REPORTS, AND PREVIOUS INSPECTION CONCERNS. OF THE THREE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN TWO AREAS. ONE ITEM OF NONCOMPLIANCE WAS IDENTIFIED IN ONE AREA (INFRACTION - FAILURE TO CONDUCT PREOPERATIONAL TESTING ON DESIGN FEATURE SPECIFIED IN THE FSAR).

REPORTS RECEIVED FROM LICENSEE

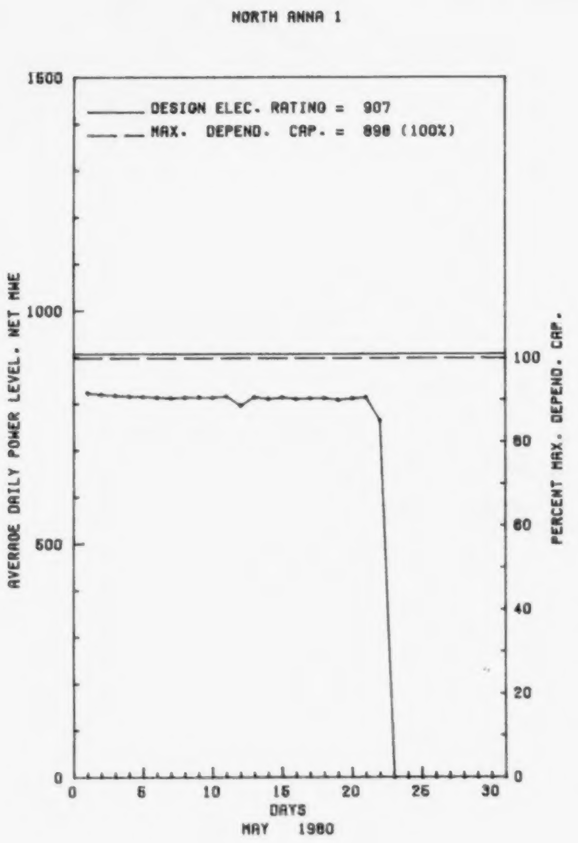
NUMBER	DATE OF. EVENT	DATE OF REPORT	TYPE OF REPORT SUBJECT
80-024/ 03L-0	03/04/80	04/02/80	ONE SAFEGUARDS AREA VENTILATION SYSTEM FILTER FOUND INOPERABLE.
80-027/ 99X-1	02/20/80	05/01/80	ABNORMAL NOISE IN STEAM GENERATORS FOLLOWING A REACTOR TRIP.
80-033/ 03L-0	03/03/80	04/03/80	VALVE POSITION PROCEDURE IN ERROR.
80-035/ 01T-0	03/13/80	03/28/80	REACTOR UPPER INTERNAL SUPPORT PLATE/GUIDE TUBE PINS MAY FATIGUE FAIL.
80-036/ 03L-0	03/30/80	04/25/80	TEST PROCEDURE ERROR RESULTED IN CONTROL ROOM BOTTLED AIR SYSTEM DEPRESSURIZING.
80-037/ 01T-0	04/03/80	04/16/80	SAFETY INJECTION ACTUATION CAUSED BY STEAM VALVE SHUTTING DURING TEST.
80-038/ 03L-0	04/09/80	05/01/80	AFD (AXIAL FLUX DIFFERENCE) DEVIATED FROM TARGET BAND BY GREATER THAN 5% FOR 3 MINUTE DURATION.

REVIEWED BY OMPA LOEB: T. C. Cintula **DATE:** 7.3.80

REVIEWED BY NRR DOR: L.B. Engle **DATE:** 7.8.80

NORTH ANNA 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-338
2. Utility Contact: W. R. Madison (703) 894-5151
3. Licensed Thermal Power (MWt): 2775
4. Nameplate Rating (Gross MWe): 1089 x 0.9 = 980
5. Design Electrical Rating (Net MWe): 907
6. Maximum Dependable Capacity (Gross MWe): 928
7. Maximum Dependable Capacity (Net MWe): 898
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	3,647.0	17,424.0
12. Hours Reactor Was Critical	526.6	2,859.8	13,025.0
13. Reactor Reserve Shutdown Hours	.0	41.9	190.2
14. Hours Generator On-Line	526.6	2,703.0	12,756.1
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,454,808	6,865,077	33,144,990
17. Gross Electrical Energy (MWH)	454,039	2,149,851	10,496,200
18. Net Electrical Energy (MWH)	428,037	2,021,813	9,875,259
19. Unit Service Factor	70.8	74.1	73.2
20. Unit Availability Factor	70.8	74.1	73.2
21. Unit Capacity Factor (MDC Net)	64.1	61.7	63.1
22. Unit Capacity Factor (DER Net)	63.4	61.1	62.5
23. Unit Forced Outage Rate	29.2	13.2	11.1

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 06/03/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-13	05/22/80	F	217.4	A	3	80-47/01T	SF	GENERA	HI-HI STEAM GENERATOR WATER LEVEL. FEED REG. VALVE FCV-1498 FOR "C" STEAM GENERATOR FAILED OPEN. VALVE WILL BE REPAIRED.

SUMMARY

THE UNIT OPERATED NORMALLY UNTIL SHUTDOWN ON 05/22/80 DUE TO STEAM GENERATOR VALVE FAILURE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

OCONEE 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	South Carolina/Oconee 30 miles W of Greenville
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2568
5.	Design Electrical Rating (MWE Net)	887
6.	Date of Initial Criticality	April 19, 1973
7.	Date of Commercial Operation	July 15, 1973
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Keowee Lake

Utility & Contractor Information

10.	Licensee	Duke Power Company
11.	Corporate Address	422 S. Church Street Charlotte, North Carolina 28242
12.	Corporate Contact	W. O. Parker, Jr.
13.	Architect/Engineer	DPC and Bechtel (Containment Bldg.)
14.	Nuclear Steam Supply System	B&W
15.	Constructor	DPC

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R.C. Lewis
18.	IE Section Chief	R. MARTIN
19.	IE Resident Inspector	F. Jape (803) 882-6927
20.	Licensing Project Manager	M. B. Fairtile
21.	Docket Number	50-269
22.	License Number/Date of Issue	DPR-38, 2/6/73
23.	Public Document Room	Oconee County Library 201 S. Spring Street Walhalla, South Carolina 29691

INSPECTION STATUS

ENFORCEMENT STATUS

AS REQUIRED BY TECHNICAL SPECIFICATION 6.2.6.1.B(3) A WRITTEN REPORT IS REQUIRED TO BE SUBMITTED WITHIN 30 DAYS OF DISCOVERY OF AN INADEQUACY IN THE IMPLEMENTATION OF PROCEDURAL CONTROLS WHICH ARE INTENDED TO ENSURE SURVEILLANCE ACTIVITIES ARE PERFORMED WITHIN SPECIFIED TIME INTERVALS TO MEET OPERABILITY REQUIREMENTS OF LIMITING CONDITIONS FOR OPERATIONS. CONTRARY TO THE ABOVE, ON FEBRUARY 1, 1979 AND ON FEBRUARY 28, 1979, THE LICENSEE DISCOVERED THAT SURVEILLANCE TESTING ON SAFETY RELATED BATTERIES AND REACTOR BUILDING PRESSURE ANALOG CHANNELS HAD NOT BEEN PERFORMED WITHIN THE TIME INTERVAL SPECIFIED BY TECHNICAL SPECIFICATIONS 4.6 AND 4.1 RESPECTIVELY. BOTH EVENTS WERE INVESTIGATED BY THE LICENSEE BUT WERE NOT REPORTED TO THE NRC.
(8017 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

UNIT OPERATING ON THREE REACTOR COOLANT PUMPS, LIMITING POWER TO 75%.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

74% POWER.

LAST IE SITE INSPECTION DATE: MAY 1-30, 1980

INSPECTION REPORT NO: 50-269/80-20

SUMMARY

INSPECTION SUMMARY

INSPECTION APRIL 28 - MAY 1 (80-14): THIS SPECIAL, UNANNOUNCED INSPECTION INVOLVED 11 INSPECTOR-HOURS ON SITE IN THE AREAS OF FIRE PROTECTION/PREVENTION. OF THE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION APRIL 15-17 AND 21-23 (80-15): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 30.5 INSPECTOR-HOURS ON SITE IN THE AREAS OF IE BULLETINS 79-13 AND 79-17 (UNIT 2); LOW PRESSURE TURBINE ROTOR UT EXAMINATION (UNIT 2); INSERVICE INSPECTION, (UNIT 2); SAFE SHUTDOWN FACILITY PIPE WELDING. OF THE FIVE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN FOUR AREAS; ONE ITEM OF NONCOMPLIANCE WAS IDENTIFIED IN ONE AREA (INFRACTION - IMPROPERLY DESIGNED ISI CALIBRATION BLOCKS).

INSPECTION APRIL 22-25 (80-16): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 8 INSPECTOR-HOURS ON SITE IN THE AREAS OF CONTAINMENT INTEGRATED LEAKAGE RATE TESTING WHICH INCLUDED INSPECTION OF TEST PROCEDURES AND POTENTIAL PROBLEM AREAS DURING THE UNIT 2 LEAKAGE RATE TEST, AND INSPECTION OF THE PLANT SURVEILLANCE PROGRAM OF PIPE SUPPORT AND RESTRAINT SYSTEMS FOR UNITS 1, 2 AND 3. OF THE TWO AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION APRIL 1-30 (80-17): THIS ROUTINE INSPECTION INVOLVED 46 RESIDENT INSPECTOR-HOURS ON SITE IN THE AREAS OF PLANT OPERATIONS, OUTAGE ACTIVITIES, TEST WITNESSING, MAINTENANCE WITNESSING, FOLLOWUP ON PREVIOUS INSPECTION FINDINGS, COMPLIANCE WITH FEBRUARY 7 AND APRIL 17, 1980 SHOW CAUSE ORDERS AND REVIEW OF THE STATION SURVEILLANCE TESTING PROGRAM. OF THE SEVEN AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN SIX AREAS; ONE ITEM OF NONCOMPLIANCE WAS FOUND IN ONE AREA (DEFICIENCY - FAILURE TO SUBMIT THIRTY DAY REPORTS WHEN A SURVEILLANCE TEST FREQUENCY WAS MISSED).

INSPECTION MAY 13-15 (80-18): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 7 INSPECTOR-HOURS ON SITE IN THE AREAS OF STRUCTURAL CONCRETE FOR THE STANDBY SHUTDOWN FACILITY. OF THE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

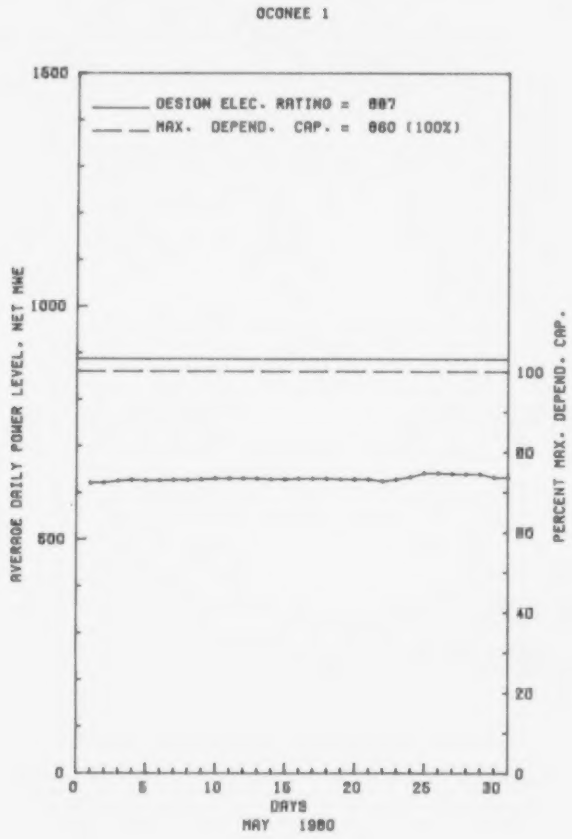
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-007/03L-0	03/15/80	04/14/80	BREACH DISCOVERED IN CABLE ROOM PENETRATION FIRE BARRIER.
80-008/15T-0	03/27/80	04/10/80	ENVIRONMENTALLY UNQUALIFIED VALVE OPERATOR.
80-009/03L-0	04/12/80	05/08/80	FIRE DETECTION SYSTEM INOPERABLE.

REVIEWED BY OMPA LOEB: P E Bole **DATE:** 7/3/80

REVIEWED BY NRR DOR: M. B. Fairtile **DATE:** 7/7/80

OCONEE 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-269
2. Utility Contact: J. A. Reavis (704) 573-8552
3. Licensed Thermal Power (MWt): 2568
4. Nameplate Rating (Gross MWe): 1038 x 0.9 = 934
5. Design Electrical Rating (Net MWe): 887
6. Maximum Dependable Capacity (Gross MWe): 899
7. Maximum Dependable Capacity (Net MWe): 860
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

CAPACITY FACTORS CALCULATED WITH A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	3,647.0	60,288.0
12. Hours Reactor Was Critical	744.0	2,321.5	42,836.2
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	744.0	2,256.8	40,203.9
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,340,996	4,778,770	93,996,889
17. Gross Electrical Energy (MWH)	491,810	1,721,790	32,636,090
18. Net Electrical Energy (MWH)	469,763	1,633,066	30,864,565
19. Unit Service Factor	100.0	61.9	66.7
20. Unit Availability Factor	100.0	61.9	66.7
**21. Unit Capacity Factor (MDC Net)	73.4	52.1	59.3
22. Unit Capacity Factor (DER Net)	71.2	50.5	57.7
23. Unit Forced Outage Rate	.0	12.6	17.7

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
** REACT COOL PUMP 6/27/80; INST HPCI CHECK VLV TMI MOD 9/5/80.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
6P	05/01/80	F	.0	A	5		CB	MOTORX	1 "B" REACTOR COOLANT PUMP IS OUT OF SERVICE. WILL REQUIRE OUTAGE FOR INSPECTION BEFORE RESTARTING.

SUMMARY

THE UNIT RAN AT REDUCED POWER DUE TO 1B1 REACTOR COOLANT PUMP BEING OUT OF SERVICE FOR THE MONTH.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source

OCONEE 2

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	South Carolina/Oconee 30 miles W of Greenville
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2568
5.	Design Electrical Rating (MWE Net)	887
6.	Date of Initial Criticality	November 11, 1973
7.	Date of Commercial Operation	September 9, 1974
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Keowee Lake

Utility & Contractor Information

10.	Licensee	Duke Power Company
11.	Corporate Address	422 S. Church Street Charlotte, North Carolina 28242
12.	Corporate Contact	W. O. Parker, Jr.
13.	Architect/Engineer	DPC/Bechtel
14.	Nuclear Steam Supply System	B&W
15.	Constructor	DPC

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R.C. Lewis
18.	IE Section Chief	R. MARTIN
19.	IE Resident Inspector	F. Jape (803) 882-6927
20.	Licensing Project Manager	M. B. Fairtile
21.	Docket Number	50-270
22.	License Number/Date of Issue	DPR-47, 10/6/73
23.	Public Document Room	Oconee County Library 201 S. Spring Street Walhalla, South Carolina 29691

INSPECTION STATUS

ENFORCEMENT STATUS

AS REQUIRED BY (G)(4) OF 10 CFR 50.55(A) AND IMPLEMENTED BY THE TECHNICAL SPECIFICATION, PARAGRAPH 4.2, AS UPDATED, REQUIRES THE SURVEILLANCE PROGRAM OF CLASS 1, CLASS 2 ... COMPONENTS COMPLY WITH ASME SECTION XI (74S75). PARAGRAPHS I-3131 AND I-3210 REQUIRE THAT BASIC CALIBRATION BLOCK CONFIGURATION REFLECTOR SIZE AND LOCATION COMPLY WITH FIG. I-3131. CONTRARY TO THE ABOVE REQUIREMENT, ON APRIL 15, 1980, THE SIZE AND LOCATION OF HOLES AND NOTCHES (REFLECTORS) ON ISI BASIC CALIBRATION BLOCKS NOS. 40303 AND 40304 DID NOT COMPLY WITH FIG. I-3131 OF THE ABOVE ASME SECTION XI CODE.
(8010 INFRACTION)

AS REQUIRED BY TECHNICAL SPECIFICATION 6.6.2.1.B(3) A WRITTEN REPORT IS REQUIRED TO BE SUBMITTED WITHIN 30 DAYS OF DISCOVERY OF AN INADEQUACY IN THE IMPLEMENTATION OF PROCEDURAL CONTROLS WHICH ARE INTENDED TO ENSURE SURVEILLANCE ACTIVITIES ARE PERFORMED WITHIN SPECIFIED TIME INTERVALS TO MEET OPERABILITY REQUIREMENTS OF LIMITING CONDITIONS FOR OPERATIONS. CONTRARY TO THE ABOVE, ON FEBRUARY 1, 1979 AND ON FEBRUARY 28, 1979, THE LICENSEE DISCOVERED THAT SURVEILLANCE TESTING ON SAFETY RELATED BATTERIES AND REACTOR BUILDING PRESSURE ANALOG CHANNELS HAD NOT BEEN PERFORMED WITHIN THE TIME INTERVAL SPECIFIED BY TECHNICAL SPECIFICATIONS 4.6 AND 4.1 RESPECTIVELY. BOTH EVENTS WERE INVESTIGATED BY THE LICENSEE BUT WERE NOT REPORTED TO THE NRC.
(8012 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

ANTICIPATED CRITICALITY FOLLOWING OUTAGE 6/14/80.

MANAGERIAL ITEMS:

NONE

PLANT STATUS:

REFUELING OUTAGE AND PLANNED MAINTENANCE.

LAST IE SITE INSPECTION DATE: MAY 28-30, 1980

INSPECTION REPORT NO: 50-270/80-16

SUMMARY

INSPECTION SUMMARY

INSPECTION APRIL 28 - MAY 1 (80-09): THIS SPECIAL, UNANNOUNCED INSPECTION INVOLVED 11 INSPECTOR-HOURS ON SITE IN THE AREAS OF FIRE PROTECTION/PREVENTION. OF THE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION APRIL 15-17 AND 21-23 (80-10): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 30 INSPECTOR-HOURS ON SITE IN THE AREAS OF IE BULLETINS 79-13 AND 79-17 (UNIT 2); LOW PRESSURE TURBINE ROTOR UT EXAMINATION (UNIT 2); INSERVICE INSPECTION, (UNIT 2); SAFE SHUTDOWN FACILITY PIPE WELDING. OF THE FIVE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN FOUR AREAS; ONE ITEM OF NONCOMPLIANCE WAS IDENTIFIED IN ONE AREA (INFRACTION - IMPROPERLY DESIGNED ISI CALIBRATION BLOCKS).

INSPECTION APRIL 22-25 (80-11): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 9 INSPECTOR-HOURS ON SITE IN THE AREAS OF CONTAINMENT INTEGRATED LEAKAGE RATE TESTING WHICH INCLUDED INSPECTION OF TEST PROCEDURES AND POTENTIAL PROBLEM AREAS DURING THE UNIT 2 LEAKAGE RATE TEST, AND INSPECTION OF THE PLANT SURVEILLANCE PROGRAM OF PIPE SUPPORT AND RESTRAINT SYSTEMS FOR UNITS 1, 2 AND 3. OF THE TWO AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION APRIL 1-30 (80-12): THIS ROUTINE INSPECTION INVOLVED 46 RESIDENT INSPECTOR-HOURS ON SITE IN THE AREAS OF PLANT OPERATIONS, OUTAGE ACTIVITIES, TEST WITNESSING, MAINTENANCE WITNESSING, FOLLOWUP ON PREVIOUS INSPECTION FINDINGS, COMPLIANCE WITH FEBRUARY 7 AND APRIL 17, 1980 SHOW CAUSE ORDERS AND REVIEW OF THE STATION SURVEILLANCE TESTING PROGRAM. OF THE SEVEN AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN SIX AREAS; ONE ITEM OF NONCOMPLIANCE WAS FOUND IN ONE AREA (DEFICIENCY - FAILURE TO SUBMIT THIRTY DAY REPORTS WHEN A SURVEILLANCE TEST FREQUENCY WAS MISSED).

INSPECTION MAY 13-15 (80-13): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 7 INSPECTOR-HOURS ON SITE IN THE AREAS OF STRUCTURAL CONCRETE FOR THE STANDBY SHUTDOWN FACILITY. OF THE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

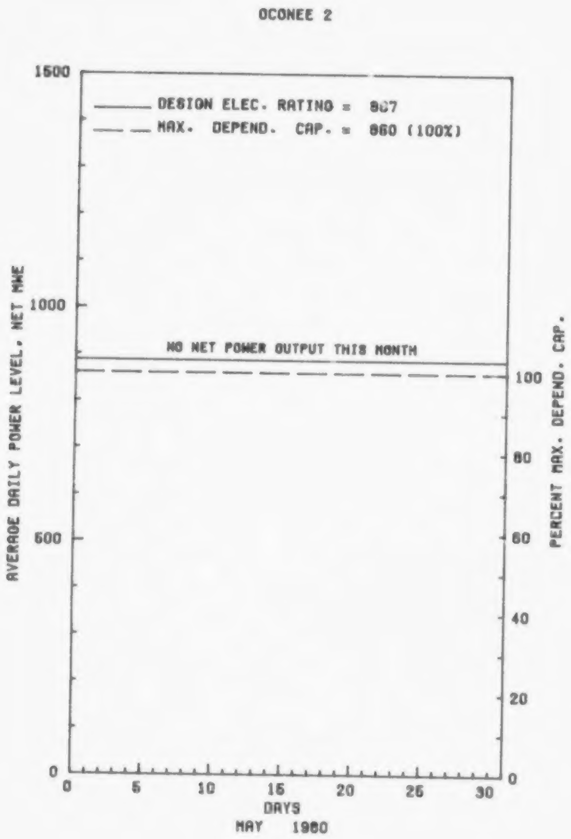
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
NONE				

REVIEWED BY OMPA LOEB: K E Bobe **DATE:** 7/3/80

REVIEWED BY NRR DOR: M. B. Fairtile **DATE:** 7/7/80

OCONEE 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-270
2. Utility Contact: J. A. Reavis (704) 373-8552
3. Licensed Thermal Power (MWt): 2568
4. Nameplate Rating (Gross MWe): 1038 x 0.9 = 934
5. Design Electrical Rating (Net MWe): 887
6. Maximum Dependable Capacity (Gross MWe): 899
7. Maximum Dependable Capacity (Net MWe): 860

CAPACITY FACTORS CALCULATED WITH A WEIGHTED AVERAGE

8 If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	3,647.0	50,208.0
12. Hours Reactor Was Critical	.0	1,536.3	35,132.3
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	.0	1,521.8	34,297.9
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	3,326,000	81,373,404
17. Gross Electrical Energy (MWH)	0	1,126,140	27,640,496
18. Net Electrical Energy (MWH)	-2,229	1,065,314	26,229,072
19. Unit Service Factor	.0	41.7	68.3
20. Unit Availability Factor	.0	41.7	68.3
21. Unit Capacity Factor (MDC Net)	.0	34.0	60.4
22. Unit Capacity Factor (DER Net)	.0	32.9	58.9
23. Unit Forced Outage Rate	.0	.0	19.3

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
TMI RELATED MODIFICATION - OCTOBER 31, 1980 - 4 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: 06/23/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
2A	03/04/80	S	744.0	D	4		XX	PIPEXX	REFUELING HAS BEEN COMPLETED AND PIPE HANGER/SUPPORT INSPECTION AND MODIFICATION (IE BULLETIN 79-02 AND 79-14) CONTINUE WITH OTHER NSM'S AND PLANNED MAINTENANCE.

SUMMARY

THE UNIT WAS SHUTDOWN FOR REFUELING AND PIPE/HANGER SUPPORT MODIFICATIONS.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

OCONEE 3

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	South Carolina/Oconee 30 miles W of Greenville
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2568
5.	Design Electrical Rating (MWE Net)	887
6.	Date of Initial Criticality	September 5, 1974
7.	Date of Commercial Operation	December 16, 1974
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Keowee Lake

Utility & Contractor Information

10.	Licensee	Duke Power Company
11.	Corporate Address	422 S. Church Street Charlotte, North Caroline 28242
12.	Corporate Contact	W. O. Parker, Jr.
13.	Architect/Engineer	DPC/Bechtel
14.	Nuclear Steam Supply System	B&W
15.	Constructor	DPC

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R.C. Lewis
18.	IE Section Chief	R. MARTIN
19.	IE Resident Inspector	F. Jape (803) 882-6927
20.	Licensing Project Manager	M. B. Fairtile
21.	Docket Number	50-287
22.	License Number/Date of Issue	DPR-55, 7/19/74
23.	Public Document Room	Oconee County Library 201 S. Spring Street Walhalla, South Carolina 29691

INSPECTION STATUS

ENFORCEMENT STATUS

AS REQUIRED BY TECHNICAL SPECIFICATION 6.6.2.1.B(3) A WRITTEN REPORT IS REQUIRED TO BE SUBMITTED WITHIN 30 DAYS OF DISCOVERY OF AN INADEQUACY IN THE IMPLEMENTATION OF PROCEDURAL CONTROLS WHICH ARE INTENDED TO ENSURE SURVEILLANCE ACTIVITIES ARE PERFORMED WITHIN SPECIFIED TIME INTERVALS TO MEET OPERABILITY REQUIREMENTS OF LIMITING CONDITIONS FOR OPERATIONS. CONTRARY TO THE ABOVE, ON FEBRUARY 1, 1979 AND ON FEBRUARY 28, 1979, THE LICENSEE DISCOVERED THAT SURVEILLANCE TESTING ON SAFETY RELATED BATTERIES AND REACTOR BUILDING PRESSURE ANALOG CHANNELS HAD NOT BEEN PERFORMED WITHIN THE TIME INTERVAL SPECIFIED BY TECHNICAL SPECIFICATIONS 4.6 AND 4.1 RESPECTIVELY. BOTH EVENTS WERE INVESTIGATED BY THE LICENSEE BUT WERE NOT REPORTED TO THE NRC. (8011 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

100% POWER.

LAST IE SITE INSPECTION DATE: MAY 1-30, 1980

INSPECTION REPORT NO: 50-287/80-14

SUMMARY

INSPECTION SUMMARY

INSPECTION APRIL 28 - MAY 1 (80-09): THIS SPECIAL, UNANNOUNCED INSPECTION INVOLVED 11 INSPECTOR-HOURS ON SITE IN THE AREAS OF FIRE PROTECTION/PREVENTION. OF THE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION APRIL 22-25 (80-10): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 9 INSPECTOR-HOURS ON SITE IN THE AREAS OF CONTAINMENT INTEGRATED LEAKAGE RATE TESTING WHICH INCLUDED INSPECTION OF TEST PROCEDURES AND POTENTIAL PROBLEM AREAS DURING THE UNIT 2 LEAKAGE RATE TEST, AND INSPECTION OF THE PLANT SURVEILLANCE PROGRAM OF PIPE SUPPORT AND RESTRAINT SYSTEMS FOR UNITS 1, 2 AND 3. OF THE TWO AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION APRIL 1-30 (80-11): THIS ROUTINE INSPECTION INVOLVED 46 RESIDENT INSPECTOR-HOURS ON SITE IN THE AREAS OF PLANT OPERATIONS, OUTAGE ACTIVITIES, TEST WITNESSING, MAINTENANCE WITNESSING, FOLLOWUP ON PREVIOUS INSPECTION FINDINGS, COMPLIANCE WITH FEBRUARY 7 AND APRIL 17, 1980 SHOW CAUSE ORDERS AND REVIEW OF THE STATION SURVEILLANCE TESTING PROGRAM. OF THE SEVEN AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN SIX AREAS; ONE ITEM OF NONCOMPLIANCE WAS FOUND IN ONE AREA (DEFICIENCY - FAILURE TO SUBMIT THIRTY DAY REPORTS WHEN A SURVEILLANCE TEST FREQUENCY WAS MISSED).

INSPECTION MAY 13-15 (80-12): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 6 INSPECTOR-HOURS ON SITE IN THE AREAS OF STRUCTURAL CONCRETE FOR THE STANDBY SHUTDOWN FACILITY. OF THE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

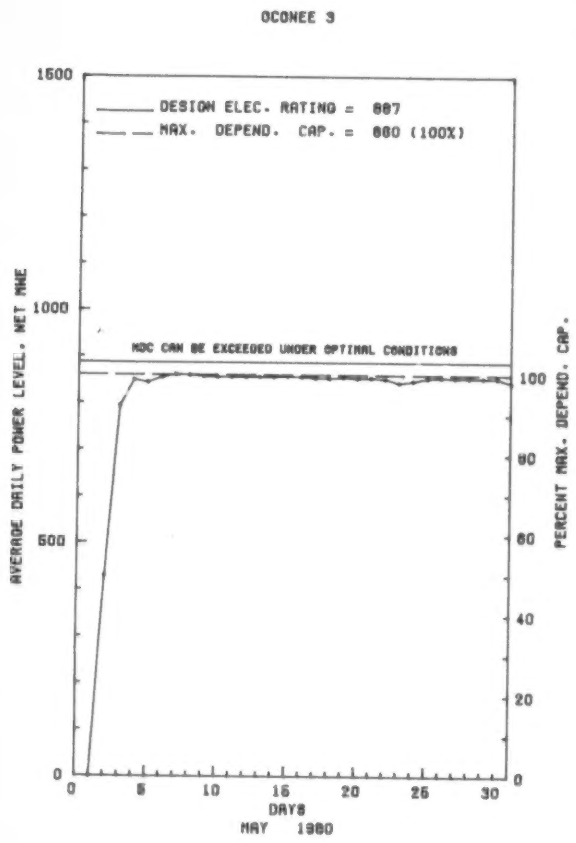
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
	NONE			

REVIEWED BY OMPA LOEB: Bobe **DATE:** 7/3/80

REVIEWED BY NRR DOR: M. B. Fairtile **DATE:** 7/7/80

OCONEE 3

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-287
2. Utility Contact: J. A. Reavis (704) 373-8552
3. Licensed Thermal Power (MWt): 2568
4. Nameplate Rating (Gross MWe): 1038 x 0.9 = 934
5. Design Electrical Rating (Net MWe): 887
6. Maximum Dependable Capacity (Gross MWe): 899
7. Maximum Dependable Capacity (Net MWe): 860
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

CAPACITY FACTORS CALCULATED WITH A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	3,647.0	47,855.0
12. Hours Reactor Was Critical	744.0	2,533.6	34,427.5
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	720.1	2,486.2	33,548.1
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,821,724	6,321,994	80,703,599
17. Gross Electrical Energy (MWH)	629,460	2,180,070	27,931,336
18. Net Electrical Energy (MWH)	602,076	2,078,632	26,575,188
19. Unit Service Factor	96.8	68.2	70.1
20. Unit Availability Factor	96.8	68.2	70.1
21. Unit Capacity Factor (MDC Net)	94.1	66.3	64.3
22. Unit Capacity Factor (DER Net)	91.2	64.3	62.6
23. Unit Forced Outage Rate	3.2	6.9	17.4

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUELING - DECEMBER 5, 1980 - 15 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
10	05/01/80	F	23.9	G	1		HA	GENERA	FAILURE TO REMOVE GROUND STRAPS ON GENERATOR PHASES BEFORE CLOSING THE EXCITER FIELD BREAKER NECESSITATED INSPECTION OF BUS BEFORE PUTTING UNIT IN SERVICE.

SUMMARY

FOLLOWING BEGINNING OF THE MONTH OUTAGE DUE TO GENERATOR INSPECTION, THE UNIT RAN AT NORMAL POWER LEVELS.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

OYSTER CREEK

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	New Jersey/Ocean 9 miles S of Toms River
2.	Electric Reliability Council	Mid-Atlantic Area Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	1930
5.	Design Electrical Rating (MWE Net)	650
6.	Date of Initial Criticality	May 3, 1969
7.	Date of Commercial Operation	December 1969
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Barnegat Bay

Utility & Contractor Information

10.	Licensee	Jersey Central Power & Light Co.
11.	Corporate Address	Madison Avenue At Punch Bowl Road Morristown, New Jersey 07960
12.	Corporate Contact	I. Finfrock, Vice President Power Generation
13.	Architect/Engineer	Burns & Roe
14.	Nuclear Steam Supply System	GE
15.	Constructor	J. A. Jones

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	IE Section Chief	R. R. KEIMIG
19.	IE Resident Inspector	L. BRIGGS (X5306) (609) 693-0702 NRC RESIDENT OFFICE
20.	Licensing Project Manager	W. A. Paulson
21.	Docket Number	50-219
22.	License Number/Date of Issue	DPR-16, 4/9/69
23.	Public Document Room	Ocean County Library 15 Hooper Avenue Toms River, New Jersey 08753

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS:

The plant is shutdown for refueling with return to operation scheduled for about June 7, 1980.

LAST IE SITE INSPECTION DATE: 5/30/80

INSPECTION REPORT NO: 50-219/80-23

SUMMARY

INSPECTION SUMMARY

50-219/80-03 - Jan 21-25: Routine, unannounced inspection by two regionally-based inspectors (86 hrs) of the Radiation Protection Program during refueling including: training; advanced planning and preparation; qualification of personnel; exposure control; radiation protection procedures; radioactive and contaminated material control, plant tours and followup on previous enforcement action. Areas where work was being conducted were examined to review radiation safety control procedures and practices. Four items of noncompliance were identified (Failure to perform evaluation as required by 10 CFR 20.201b sufficient to ensure compliance with 10 CFR 20.202; Failure to use respiratory protection equipment; Failure to adhere to procedures; and Failure to label radioactive material).

50-219/80-10 - Mar 17-20: Routine, unannounced inspection by a regionally-based inspector (32 hrs) of the containment integrated leak rate test procedure; containment penetration leak rate test activities; inservice testing of safety related pumps and valves; and licensee action on previous inspection findings. Two items of noncompliance (Failure to implement an inservice testing program; and Failure to implement procedures).

50-219/80-13 - Apr 7-11: Routine, unannounced inspection by one regionally-based inspector, a resident inspector and an NRC headquarters based engineer, of the Quality Assurance Program (QAP) implementation including: QA Program Review; QA/QC Administration; Design Changes/Modifications; and followup on previously identified items. The inspection involved 76 inspector-hours onsite and 46 inspector-hours at the Corporate Offices by a region-based inspector, a resident inspector and and NRC headquarters based engineer (NRR-QAB). No items of noncompliance were identified.

50-219/80-14 - Apr 18: Special, announced inspection by one regionally-based inspector (8 hrs) of the status of licensee radiation protection commitments made to NRC Region I during the period October 1978 through April 18, 1980. No items of noncompliance were identified.

50-219/80-18 - Apri 29: Special, enforcement conference convened by Region I management to discuss NRC concerns and licensee corrective actions arising from the NRC's Performance Appraisal Branch (PAB) inspection findings, radiation protection concerns resulting from recent Region I inspections, and actions to improve overall management controls. A forthcoming inspection by the Health Physics Appraisal Team was also discussed. The meeting was attended nu senior licensee and Region I management and involved three hours in the Region I office by all attendees.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

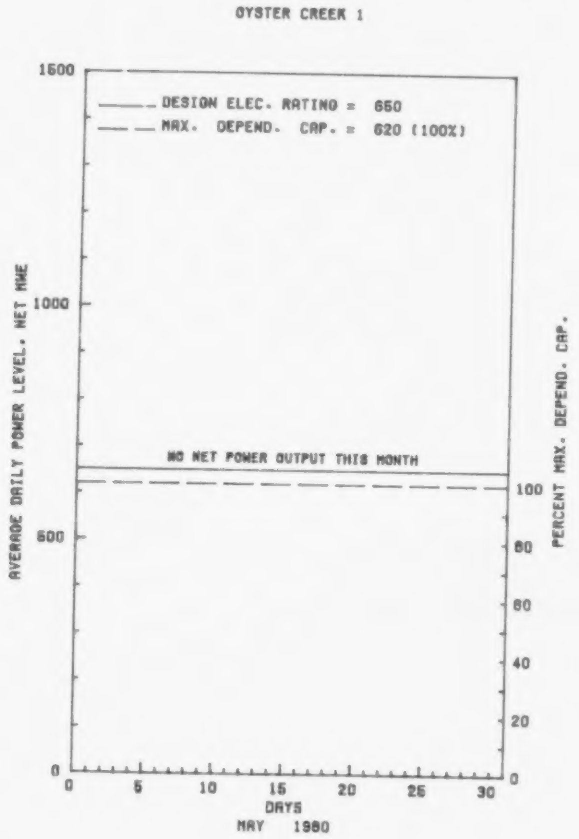
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-015/03L	04/03/80	05/07/80	Reactor building automatic isolation valve inoperative (1 of 2 in series) due to broken piston rod eye stud

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 7-3-80

REVIEWED BY NRR DOR: J J Shea **DATE:** 7-8-80

OYSTER CREEK

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-219
2. Utility Contact: C. M. McClain (201) 455-8748
3. Licensed Thermal Power (MWt): 1930
4. Nameplate Rating (Gross MWe): 687.5 x 0.8 = 550
5. Design Electrical Rating (Net MWe): 650
6. Maximum Dependable Capacity (Gross MWe): 650
7. Maximum Dependable Capacity (Net MWe): 620

ITEM 21 CALCULATED BY USING A WEIGHTED AVERAGE

8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	3,647.0	91,511.0
12. Hours Reactor Was Critical	.0	101.5	68,740.8
13. Reactor Reserve Shutdown Hours	.0	.0	468.2
14. Hours Generator On-Line	.0	98.4	67,403.9
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	157,611	114,036,769
17. Gross Electrical Energy (MWH)	0	53,670	38,935,985
18. Net Electrical Energy (MWH)	0	48,612	37,529,625
19. Unit Service Factor	.0	2.7	73.7
20. Unit Availability Factor	.0	2.7	73.7
21. Unit Capacity Factor (MDC Net)	.0	2.1	67.8
22. Unit Capacity Factor (DER Net)	.0	2.1	63.1
23. Unit Forced Outage Rate	.0	.0	6.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 06/11/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	01/05/80	S	744.0	C	4		RC	FUELXX	REFUELING/MAINTENANCE OUTAGE.

SUMMARY

THE UNIT REMAINED SHUTDOWN ALL MONTH FOR MAINTENANCE AND REFUELING.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source

PALISADES

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1. Location (state/county) (nearest population center)	Michigan/Van Buren 5 miles S of South Haven
2. Electric Reliability Council	East Central Area Reliability Coordination Agreement
3. Type of Reactor	PWR
4. Power Level (MWT)	2530
5. Design Electrical Rating (MWE Net)	805
6. Date of Initial Criticality	May 24, 1971
7. Date of Commercial Operation	December 31, 1971
8. Condenser Cooling Method	Forced Draft Cooling Towers
9. Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10. Licensee	Consumers Power Company
11. Corporate Address	212 W. Michigan Avenue Jackson, Michigan 49201
12. Corporate Contact	R. B. DeWitt
13. Architect/Engineer	Bechtel
14. Nuclear Steam Supply System	Combustion Engineering
15. Constructor	Bechtel

Regulatory Information

16. IE Region responsible	III
17. IE Branch Chief	R. Heishman
18. IE Section Chief	D.C. Boyd
19. IE Resident Inspector	B. Jorgenson (616) 764-8971
20. Licensing Project Manager	T. Wambach
21. Docket Number	50-255
22. License Number/Date of Issue	DPR-20, 3/24/71
23. Public Document Room	Kalamazoo Public Library 315 South Rose Street Reference Department Kalamazoo, Michigan 49006

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENTS

Facility modifications pursuant to NUREG-0578 have been completed. Other significant modifications also completed include blank-flanging containment ventilation supply and exhaust 48 inch lines; closing main-feed regulating and bypass valves on low steam-generator pressure; and functionally separating HPSI injection to two independent trains. (80-06)

FACILITY ITEMS

Extensive reviews and modifications to facility procedures in nearly all categories; particulary administrative, operating, off-normal, emergency and surveillance testing procedures, have been completed pursuant to NRC Bulletins and Orders and to address system and modifications. (80-06)

MANAGERIAL ITEMS

Mr. William Skibitsky assumed duties as the plant Operations Superintendent effective March 31, 1980. (80-05)

PLANT STATUS

Reactor resumed operation at 0544 on May 24, 1980, concluding a refueling and maintenance outage which began on September 8, 1979.

LAST IE SITE INSPECTION DATE: April 23, 1980

INSPECTION REPORT NO: 80-07

SUMMARY

INSPECTION SUMMARY

Inspection on March 25-27, (80-04): Included a review of the licensee's Security Plan as it related to alleged security concerns expressed by members of the guard force. The inspection involved 42 inspector hours onsite by two NRC inspectors. The inspection was begun during a day shift period; 10 inspector hours were accomplished during offshift periods. The licensee was found to be in compliance with NRC requirements within the areas examined during this inspection, except for the following item: Physical Barriers - A secondary physical barrier did not provide adequate protection.

Inspection during March 1980, (80-05): Resident inspection program activities: operations, reportable events, IE Bulletins, NUREG-0578 requirements; and followup of previously identified open inspection items. Special inspection activities: review of licensee actions on small break loss of coolant (SBLOCA) accident requirements. The inspection involved 176 inspector hours onsite by two NRC inspectors. No items of noncompliance or deviations were identified in any of the areas inspected.

Inspection on April 1 through May 2, (80-06): Routine resident inspection program activities including activities during long-term shutdown; cleanliness; preparations for startup; and action on previously identified items. Special inspections of licensee activities relating to the NRC Order Modifying License DPR-20 dated November 9, 1979 and activities relating to coping with small-break loss of coolant accidents. The inspections involved 186 inspector hours onsite by four NRC inspectors. No items of noncompliance or deviations were identified in any of the areas inspected.

Investigation on April 23, (80-07): Allegations concerning poor weld quality of rod hangers on the containment spray ring and also of the support restraints No. H896 and No. H897. The investigation involved five investigation hours onsite by one NRC investigator. No items of noncompliance or deviations were identified

REPORTS RECEIVED FROM LICENSEE

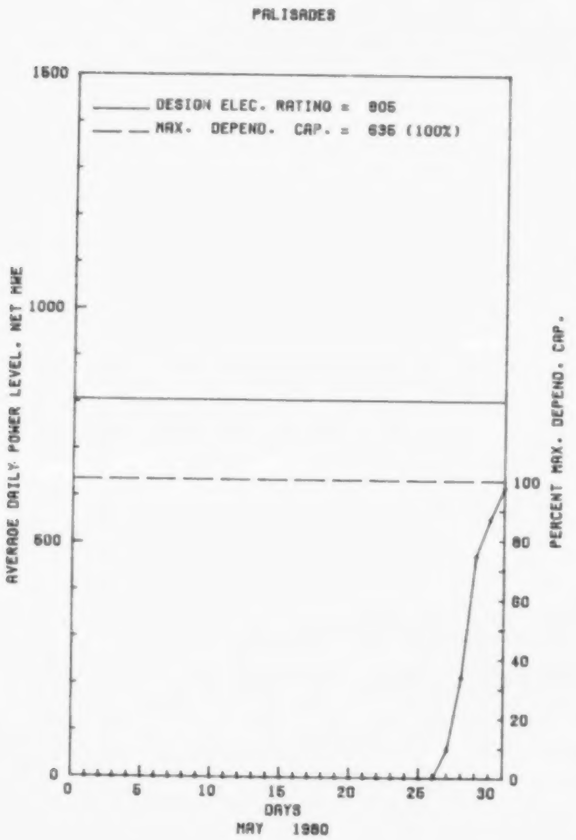
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-09/ 01T-0	04/24/[illegible]	05/07/80	During walkdowns of safety related systems conducted to verify P&ID accuracy, the ventilation supply damper (CV-1812) to the west safeguards equipment room could not be located.
80-10/ 01T-0	03/26/80	05/07/80	During review of TS Surveillance Procedures, it was determined that during calibration of the SIRW tank low level switches in 3/76, the RAS setpoint exceeded the TS limit of 27 inches above the tank bottom.
80-11/ 03L-0	04/16/80	05/14/80	Following modifications to PORV supplies the valves were stroked per pre-op test procedure and declared operable.
80-12/ 03L-0	04/17/80	05/15/80	During inspection of fire barriers, it was discovered that the bottom half of the barrier for fire prevention HO-12 was missing.

REVIEWED BY OMPA LOEB: Bode DATE: 7/3/80

REVIEWED BY NRR DOR: T. V. Wambach DATE: 7/8/80

PALISADES

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-255
2. Utility Contact: P. J. Bodtke (616) 764-8913
3. Licensed Thermal Power (MWt): 2530
4. Nameplate Rating (Gross MWe): 955 x 0.85 = 812
5. Design Electrical Rating (Net MWe): 805
6. Maximum Dependable Capacity (Gross MWe): 675
7. Maximum Dependable Capacity (Net MWe): 635
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	3,647.0	74,078.0
12. Hours Reactor Was Critical	180.6	180.6	39,745.4
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	102.6	102.6	37,529.1
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	161,088	161,088	71,672,568
17. Gross Electrical Energy (MWH)	50,730	50,730	22,082,960
18. Net Electrical Energy (MWH)	46,188	46,188	20,716,963
19. Unit Service Factor	13.8	2.8	50.7
20. Unit Availability Factor	13.8	2.8	50.7
21. Unit Capacity Factor (MDC Net)	9.8	2.0	44.0
22. Unit Capacity Factor (DER Net)	7.7	1.6	34.7
23. Unit Forced Outage Rate	.0	.0	36.1

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
13	09/08/79	S	640.5	D	4				TMI MODIFICATIONS AND ANCHOR BOLT INSPECTIONS/REPAIRS ACCORDING TO I&E BULLETIN 79-02.
14	05/28/80	S	.9	B	1				TURBINE OVERSPEED TRIP TEST.

SUMMARY

THE UNIT WAS IN A REFUELING AND MODIFICATION OUTAGE UNTIL 05/24/80. AFTER RETURNING TO SERVICE, THE UNIT TRIPPED ON 05/27/80 DUE TO AN RPS MALFUNCTION AND WAS OFF LINE 05/28/80 FOR TURBINE OVERSPEED TRIP TEST.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

PEACH BOTTOM 2

FACILITY DATA

Facility Description

1. Location (state/countv) (nearest population center)	Pennsylvania/York 19 miles S of Lancaster
2. Electric Reliability Council	Mid-Atlantic Area Council
3. Type of Reactor	BWR
4. Power Level (MWT)	3293
5. Design Electrical Rating (MWE Net)	1065
6. Date of Initial Criticality	September 16, 1973
7. Date of Commercial Operation	July 5, 1974
8. Condenser Cooling Method	Once Through with Cooling Towers
9. Condenser Cooling Water Source	Conowingo Pond/Susquehanna River

Utility & Contractor Information

10. Licensee	Philadelphia Electric Company
11. Corporate Address	2301 Market Street Philadelphia, Pennsylvania 19101
12. Corporate Contact	S. L. Daltroff, Vice President Electric Production
13. Architect/Engineer	Bechtal
14. Nuclear Steam Supply System	GE
15. Constructor	Bechtel

Regulatory Information

16. IE Region responsible	I
17. IE Branch Chief	E. Brunner
18. IE Section Chief	E. McCabe
19. IE Resident Inspector	C. Cowgill 717-456-7614
20. Licensing Project Manager	D. Verrelli
21. Docket Number	50-277
22. License Number/Date of Issue	DPR-44, 8/8/73
23. Public Document Room	Government Publications Section State Library of Pennsylvania Education Building Commonwealth and Walnut Street Harrisburg, Pennsylvania 17126

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO TITLE 10, CHAPTER 1, CODE OF FEDERAL REGULATIONS, PART 50, APP B, CRITERION V, THE LICENSEE FAILED TO FOLLOW APPROVED PROCEDURES & DID NOT PROPERLY RESPOND TO A NONCONFORMANCE REPORT. THE RESPONSE WAS MADE AFTER 96 DAYS & WAS REQUIRED TO BE MADE IMMEDIATELY WHEN THE NONCONFORMANCE WAS 30 DAYS OVERDUE.
(7909 DEFICIENCY)

CONTRARY TO TS 6.4.1, THE RETRAINING PROGRAM FOR NON-LICENSED PERSONNEL HAD NOT BEEN ESTABLISHED. IM
(7909 INFRACTION)

CONTRARY TO TS 6.5.2.8.D, NO AUDITS OF THE QA SECTION, ENGINEERING & RESEARCH DEPARTMENT OR ITS ACTIVITIES HAD BEEN PERFORMED.
(7909 INFRACTION)

CONTRARY TO THE QA PROGRAM, FSAR 17.2, APP A, THE QA INSPECTIONS WERE NOT QUALIFIED IN ACCORDANCE WITH ANSI N45.2.6-1973.
(7909 DEVIATION)

CONTRARY TO TS 6.8.1, STATION PROCEDURES A43 & A47, & TS 4.9.A.2.C, DISCHARGE TEST OF UNIT 2 125 V STATION BATTERIES WAS APPARENTLY ACCOMPLISHED IN 1978 W/O USING SURVEILLANCE TEST (ST) 8.4. DATA REQUIRED TO BE RECORDED BY TS 4.9.A.2.C & ST 8.4 WAS APPARENTLY NOT RECORDED. SATISFACTORY PERFORMANCE OF ST 8.4 WAS NOT DOCUMENTED. FOR UNITS 2 & 3, RETEST OF PREVIOUSLY UNSATISFACTORY CABLE SPREADING ROOM VENT DAMPERS WAS NOT ACCOMPLISHED PER ST 16.4 & MAINTENANCE REQUEST FORM WAS NOT USED TO DOCUMENT & CONTROL REPAIR OF VENT DAMPERS.
(8006 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

MANAGERIAL ITEMS:

The Licensee Radiation Protection Manager, W. Barler, left the Licensee Organization on May 2, 1980. A management meeting to discuss the licensee's plans for a permanent replacement is scheduled for June 3, 1980 at the NRC Region I office.

PLANT STATUS:

Shutdown for refueling and torus modification. Startup is scheduled for early July.

LAST IE SITE INSPECTION DATE: 5/31/80

INSPECTION REPORT NO: 50-277/80-15

SUMMARY

INSPECTION SUMMARY

No reports issued.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

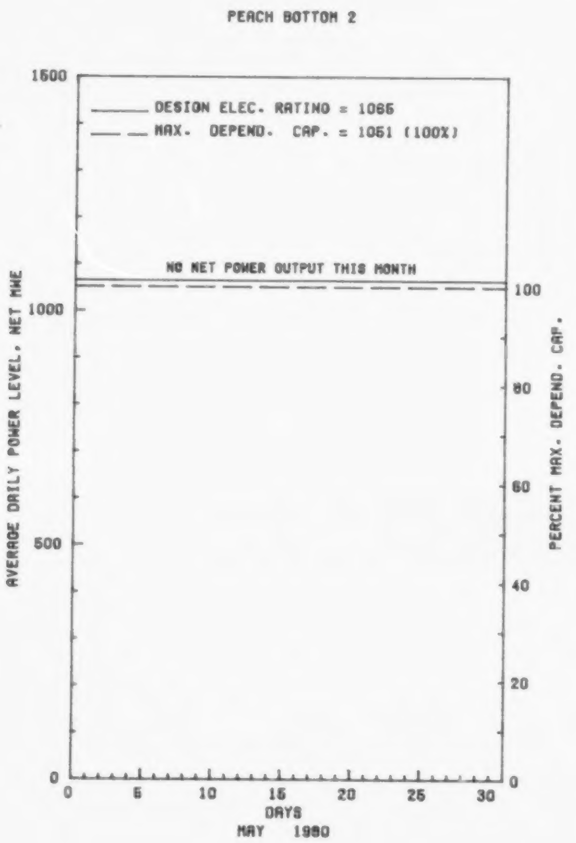
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-008/01T	04/17/80	05/01/80	Guillotine break of HPCI turbine steam supply in valve isolation room would overpressurize the room. Isol valve throttled to limit pressure transient
80-009/01T	05/02/80	05/16/80	Irradiated fuel relocated in fuel pool for inspection w/o required secondary containment integrity; fuel movement was stopped & the refueling bridge locked

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 7-8-80

REVIEWED BY NRR DOR: **DATE:** 7-7-80

PEACH BOTTOM 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-277
2. Utility Contact: W. M. Alden (215) 841-5022
3. Licensed Thermal Power (MWt): 3293
4. Nameplate Rating (Gross MWe): 1280 x 0.9 = 1152
5. Design Electrical Rating (Net MWe): 1065
6. Maximum Dependable Capacity (Gross MWe): 1098
7. Maximum Dependable Capacity (Net MWe): 1051
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	3,647.0	51,791.0
12. Hours Reactor Was Critical	.0	1,441.0	39,196.0
13. Reactor Reserve Shutdown Hours	.0	.0	0
14. Hours Generator On-Line	.0	1,390.2	38,373.0
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	3,831,770	110,597,414
17. Gross Electrical Energy (MWH)	0	1,255,980	36,314,210
18. Net Electrical Energy (MWH)	-6,242	1,189,631	34,805,416
19. Unit Service Factor	.0	38.1	74.1
20. Unit Availability Factor	.0	38.1	74.1
21. Unit Capacity Factor (MDC Net)	.0	31.0	63.9
22. Unit Capacity Factor (DER Net)	.0	30.6	63.1
23. Unit Forced Outage Rate	.0	.9	6.1

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
UNIT 2 SHUTDOWN FOR FOURTH REFUELING.
25. If Shut Down At End of Report Period, Estimated Date of Startup: 07/22/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
9	03/21/80	S	744.0	C	4		RC	FUELXX	CONTINUATION OF REFUEL OUTAGE.

SUMMARY

THE UNIT REMAINED OFF LINE IN A REFUELING OUTAGE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

PEACH BOTTOM 3

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Pennsylvania/York 19 miles S of Lancaster
2.	Electric Reliability Council	Mid-Atlantic Area Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	3293
5.	Design Electrical Rating (MWE Net)	1065
6.	Date of Initial Criticality	August 7, 1974
7.	Date of Commercial Operation	December 23, 1974
8.	Condenser Cooling Method	Once Through with Cooling Towers
9.	Condenser Cooling Water Source	Conowingo Pond/Susquehanna River

Utility & Contractor Information

10.	Licensee	Philadelphia Electric Company
11.	Corporate Address	2301 Market Street Philadelphia, Pennsylvania 19101
12.	Corporate Contact	S. L. Daltroff, Vice President Electric Production
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	GE
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	IE Section Chief	E. McCabe
19.	IE Resident Inspector	C. Cowgill (717-456-7614)
20.	Licensing Project Manager	D. Verrelli
21.	Docket Number	50-278
22.	License Number/Date of Issue	DPR-56, 7/2/74
23.	Public Document Room	Government Publications Section State Library of Pennsylvania Education Building Commonwealth and Walnut Street Harrisburg, Pennsylvania 17126

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO TITLE 10, CHAPTER 1, CODE OF FEDERAL REGULATIONS, PART 50, APP B, CRITERION V, THE LICENSEE FAILED TO FOLLOW APPROVED PROCEDURES & DID NOT PROPERLY RESPOND TO A NONCONFORMANCE REPORT. THE RESPONSE WAS MADE AFTER 96 DAYS & WAS REQUIRED TO BE MADE IMMEDIATELY WHEN THE NONCONFORMANCE WAS 30 DAYS OVERDUE.
(7910 DEFICIENCY)

CONTRARY TO TS 6.4.1, THE RETRAINING PROGRAM FOR NON-LICENSED PERSONNEL HAD NOT BEEN ESTABLISHED, IMPLEMENTED & MAINTAINED BY THE LICENSEE.
(7910 INFRACTION)

CONTRARY TO TS 6.5.2.8.D, NO AUDITS OF THE QA SECTION, ENGINEERING & RESEARCH DEPARTMENT OR ITS ACTIVITIES HAD BEEN PERFORMED.
(7910 INFRACTION)

CONTRARY TO QA PROGRAM, FSAR 17.2, APP A, THE QA INSPECTIONS WERE NOT QUALIFIED IN ACCORDANCE WITH ANSI N45.2.6-1973.
(7910 DEVIATION)

CONTRARY TO TS 6.8.1 & STATION PROCEDURES A-43 & A-47, FOR UNITS 2 & 3, RETEST OF PREVIOUSLY UNSATISFACTORY CABLE SPREADING ROOM VENT DAMPERS WAS NOT ACCOMPLISHED PER ST 16.4 & MAINTENANCE REQUEST FORM WAS NOT USED TO DOCUMENT & CONTROL REPAIRS OF VENT DAMPERS.
(8006 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

MANAGERIAL ITEMS:

The licensee Radiation Protection Manager, W. Barley, left the licensee organization on May 2, 1980. A management meeting to discuss the licensee's plans for a permanent replacement has been scheduled for June 3, 1980 at the NRC Region I office.
PLANT STATUS:

An eleven day unplanned outage began on May 3 for repairs on 3B Recirculation Pump Seal, a Low Pressure Coolant Injection System Valve, and Offgas System Piping. The unit again shutdown on May 22 due to failure of the 3B Recirculation Pump Seal and remained shutdown through the end of the month. Representative combined Unit 2 and Unit 3 Stack Gas Release Rate was 124 microcuries per second.

LAST IE SITE INSPECTION DATE: 5/31/80

INSPECTION REPORT NO: 50-278/80-13

SUMMARY

INSPECTION SUMMARY

No reports issued.

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-009/03L	04/14/80	05/14/80	Fatigue failure of 1 inch drain line connection piping on "D" RHR pump. Other heat removal systems were verified operable and repair was effected

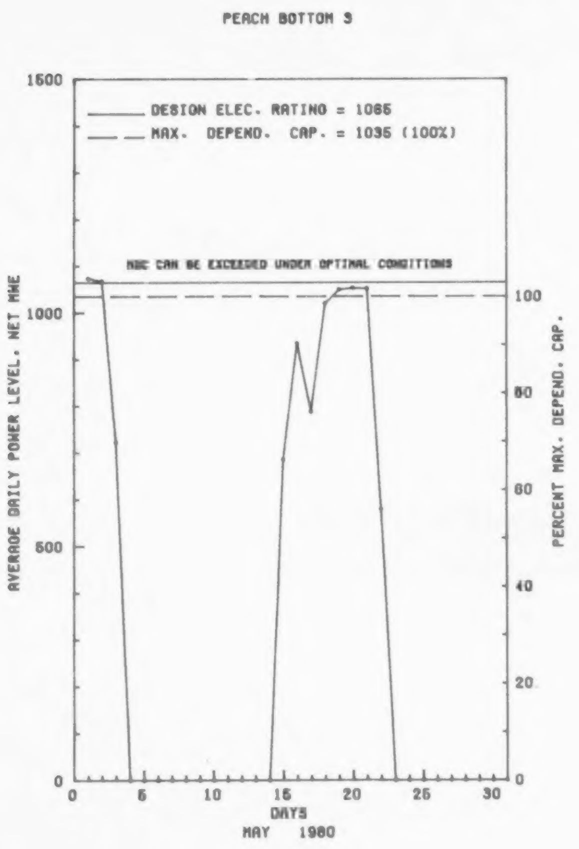
REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 7-3-80

REVIEWED BY NRR DOR: **DATE:** 7-7-80

GPO 873-667

PEACH BOTTOM 3

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-278
2. Utility Contact: W. M. Alden (215) 841-5022
3. Licensed Thermal Power (MWt): 3293
4. Nameplate Rating (Gross MWe): 1280 x 0.9 = 1152
5. Design Electrical Rating (Net MWe): 1065
6. Maximum Dependable Capacity (Gross MWe): 1098
7. Maximum Dependable Capacity (Net MWe): 1035
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	3,647.0	47,687.0
12. Hours Reactor Was Critical	277.0	2,687.0	37,533.0
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	253.1	2,556.6	36,488.4
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	758,873	8,092,083	102,311,237
17. Gross Electrical Energy (MWH)	250 380	2,720,530	33,306,390
18. Net Electrical Energy (MWH)	237,634	2,622,662	31,967,462
19. Unit Service Factor	34.0	70.1	76.5
20. Unit Availability Factor	34.0	70.1	76.5
21. Unit Capacity Factor (MDC Net)	30.9	69.5	64.8
22. Unit Capacity Factor (DER Net)	30.0	67.5	62.9
23. Unit Forced Outage Rate	66.0	20.5	8.0

24. Shutdowns Scheduled Over Next 6 Months (Type,Date,and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 06/03/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
10	05/03/80	F	265.2	A	1		CB	PUMPXX	3B REACTOR RECIRC. PUMP SEAL REPLACEMENT.
11	05/17/80	S	.0	H	5		RC	ZZZZZZ	LOAD DROP FOR CONTROL ROD ADJUSTMENT.
12	05/22/80	F	225.7	A	1		CB	PUMPXX	3B REACTOR RECIRC. PUMP SEAL REPLACEMENT.

SUMMARY

THE UNIT WAS OUT OF SERVICE FROM 05/03/80 TO 05/14/80 AND FROM 05/22/80 TO 05/31/80 FOR RECIRC PUMP SEAL REPAIR. THE REST OF THE MONTH THE PLANT RAN NEAR FULL POWER EXCEPT FOR A REDUCTION ON 05/17/80 FOR CONTROL ROD ADJUSTMENT.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

best copy available

PILGRIM

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Massachusetts/Plymouth 4 miles SE of Plymouth
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	BWR
4.	Licenses Power Level (MWT)	1998
5.	Design Electrical Rating (MWE Net)	655
6.	Date of Initial Criticality	June 16, 1972
7.	Date of Commercial Operation	December 1972
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Cape Cod Bay

Utility & Contractor Information

10.	Licensee	Boston Edison Company
11.	Corporate Address	800 Boylston Street Boston, Massachusetts 02199
12.	Corporate Contact	A. Corey, Vice President
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	GE
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	IE Section Chief	T. MARTIN
19.	IE Resident Inspector	J. JOHNSON
20.	Licensing Project Manager	M. H. Williams
21.	Docket Number	50-293
22.	License Number/Date of Issue	DPR-35, 6/8/72
23.	Public Document Room	Plymouth Public Library North Street Plymouth, Massachusetts 02360

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

Experienced HPCI/RCIC control problems during startup testing.

PLANT STATUS:

Following refueling, the reactor was taken critical on May 14, 1980 for physics testing. The plant is presently at 80% power with plans to increase to normal full power operation. Representative stack and offgas release rates are 250 and 10,000 microcuries per second, respectively.

LAST IE SITE INSPECTION DATE: 5/31/80

INSPECTION REPORT NO: 50-293/80-23

SUMMARY

INSPECTION SUMMARY

50-293/80-11 - Mar 10-14: Routine, unannounced inspection by a regionally-based inspector and an NRR engineer of the Quality Assurance Program implementation including: QAP annual review; QA/QC administration program; design changes/modification program; plant modifications; previously identified items; and, a plant tour. The inspection involved 74 inspector-hours onsite by a regionally based inspector, an NRR engineer (NRR-QAB) and a resident inspector and 52 inspector-hours at the corporate offices by a regionally based inspector, an NRR engineer, two resident inspectors and one supervisor. No items of noncompliance were identified.

50-293/80-14 - Apr 8-11: Routine, unannounced inspection by four regionally-based inspectors (78 hrs) of general employee training, licensed operator requalification training and on-the-job training for plant staff. Plant tours were made on day and evening shifts. An OSHA exit meeting was attended. One item of noncompliance was identified: (Failure to keep records as required by licensee's training manual).

50-293/80-18 - Apr 22-30: Routine, unannounced inspection by one regionally-based and resident inspector (58 hrs) of Refueling Maintenance, Licensee plans for coping with strikes, and High Energy pipe break analysis (HPCI). One item of noncompliance was identified: (Failure to control foreign material in accordance with maintenance procedure requirements).

REPORTS RECEIVED FROM LICENSEE

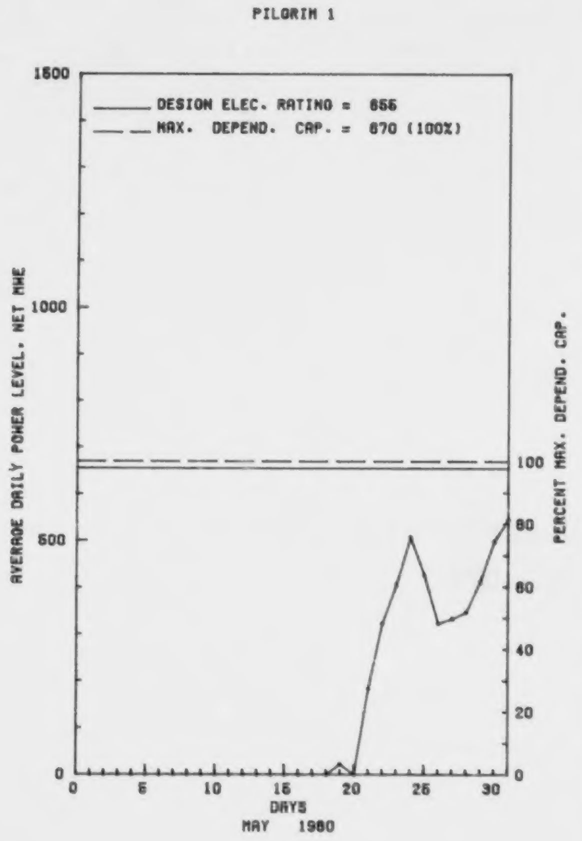
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-014/04T	04/18/80	04/28/80	Anomolous milk sample contained 10 times control valve of cesium-137
80-015/03L	04/01/80	05/01/80	Diesel generator fixed drychemical fire protection system failed during surveillance testing

REVIEWED BY OMPA LOEB: John J. Ruzz **DATE:** 7-3-80

REVIEWED BY NRR DOR: **DATE:** 7/7/80

PILGRIM

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-293
2. Utility Contact: G. G. Whitney (617) 746-7900
3. Licensed Thermal Power (MWt): 1998
4. Nameplate Rating (Gross MWe): 780 x 0.87 = 678
5. Design Electrical Rating (Net MWe): 655
6. Maximum Dependable Capacity (Gross MWe): 690
7. Maximum Dependable Capacity (Net MWe): 670
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	3,647.0	65,543.0
12. Hours Reactor Was Critical	351.9	448.0	45,428.9
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	268.2	364.3	43,916.8
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	328,056	509,208	73,600,320
17. Gross Electrical Energy (MWH)	108,530	169,870	24,334,704
18. Net Electrical Energy (MWH)	103,884	162,744	23,368,867
19. Unit Service Factor	36.0	10.0	67.0
20. Unit Availability Factor	36.0	10.0	67.0
21. Unit Capacity Factor (MDC Net)	20.8	6.7	53.2
22. Unit Capacity Factor (DER Net)	21.3	6.8	54.4
23. Unit Forced Outage Rate	8.9	6.7	10.6

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
1	01/05/80	S	449.7	C	4		RC	FUELXX	REFUELING.
2	05/19/80	F	26.1	A	3		HA	XXXXXX	REACTOR SCRAM DUE TO LOW WATER LEVEL CAUSED BY ARRATIC OPERATION OF TURBINE EPR.
3	05/26/80	F	.0	H	5		HG	DEMINX	HIGH D/P ACROSS CONDENSATE DEMINERALIZERS DUE TO RUST BUILDUP IN LINES DURING OUTAGE.

SUMMARY

THE UNIT WAS PLACED ON LINE ON 05/19/80 FOLLOWING ANNUAL REFUELING OUTAGE. IT WAS REMOVED FROM SERVICE ON 05/20/80 DUE TO TURBINE SPEED CONTROL. IT WAS RETURNED TO SERVICE ON 05/21/80 AND RAN ON A REDUCED POWER LEVEL DUE TO CONDENSATE DEMIN D/P

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

POINT BEACH 1

FACILITY DATA

Facility Description

1. Location (state/countv) (nearest population center)	Wisconsin/Manitowoc 15 miles N of Manitowoc
2. Electric Reliability Council	Mid-America Interpool Network
3. Type of Reactor	PWR
4. Power Level (MWT)	1518
5. Design Electrical Rating (MWE Net)	497
6. Date of Initial Criticality	November 2, 1970
7. Date of Commercial Operation	December 21, 1970
8. Condenser Cooling Method	Straight Through
9. Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10. Licensee	Wisconsin Electric Power Co.
11. Corporate Address	231 West Michigan Milwaukee, Wisconsin
12. Corporate Contact	S. Burstein
13. Architect/Engineer	Bechtel
14. Nuclear Steam Supply System	Westinghouse
15. Constructor	Bechtel

Regulatory Information

16. IE Region responsible	III
17. IE Branch Chief	R. Heishman
18. IE Section Chief	R. F. Warnick
19. IE Resident Inspector	J. D. SMITH
20. Licensing Project Manager	C. M. Trammell
21. Docket Number	50-266
22. License Number/Date of Issue	DPR-24 issued 10/06/70
23. Public Document Room	Document Department, Library University of Wisconsin-Stevens Point Stevens Point, Wisconsin 54481 Manitowoc Public Library Manitowoc, Wisconsin (Temporary LPDR for spent fuel pool capacity increase amendment only

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

The unit operated routinely during the month at reduced power.

LAST IE SITE INSPECTION DATE: March 10-14 and April 8, 1980

INSPECTION REPORT NO: 80-09

SUMMARY

INSPECTION SUMMARY

Inspection on March 31 through April 4, (80-06): Included a review of the Security Plan; Security Program Audit; Security Organization; Protected Area Physical Barriers; Vital Area Physical Barriers; Lighting; Access Control (Identification, Authorization and Badging); Access Control (Search); Access Control (Escorting); Access Control (Vital Areas); Alarm Stations; Detection Aids; Assessment Aids; Communications; Testing and Maintenance; Compensatory Measures; Power Supply; Response; Locks, Keys and Combinations; Records and Reports; Licensee Action of Previous Inspection Findings; Independent Inspection Effort. The inspection involved 72 inspector hours onsite by two NRC inspectors. The inspection began during the day shift; 19 inspection hours were accomplished during offshift periods. The licensee was found to be in compliance with NRC requirements within the 23 areas examined during the inspection except for the following 3 items: Vital Area Physical Barriers - Some vital area physical barriers were not properly maintained; Alarm Stations - A change in alarm status was not properly monitored; and Testing and Maintenance - Some alarm tests were not done at the required frequency.

Inspection on April 8-10, (80-07): Routine, unannounced inspection of radiation protection program associated with a refueling outage, including: radiation protection procedures; advanced planning and preparation; training; exposure control; instruments and equipment; posting and control; IE Information Notice No. 79-08; IE Circular No. 79-15; contract radiation protection technician qualifications; and specific matters concerning radioactive materials transport. The inspection involved 36 inspector hours onsite by two NRC inspectors. No items of noncompliance or deviations were identified.

Inspection on March 10-14 and April 8, (80-09): Routine, announced inspection on March 10-14, 1980 to review plant procedures, operator training, and systems utilized for Small Break Loss of Coolant event and other events affecting primary coolant inventory. On April 8, 1980 a special announced management meeting between NRC staff members, corporation management representatives of Wisconsin Electric Power, and Point Beach plant management to review plant performance was held. The inspection involved 55 inspector hours onsite by two NRC inspectors. The meeting involved 15 manhours by five NRC representatives. No items of noncompliance or deviations were identified.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

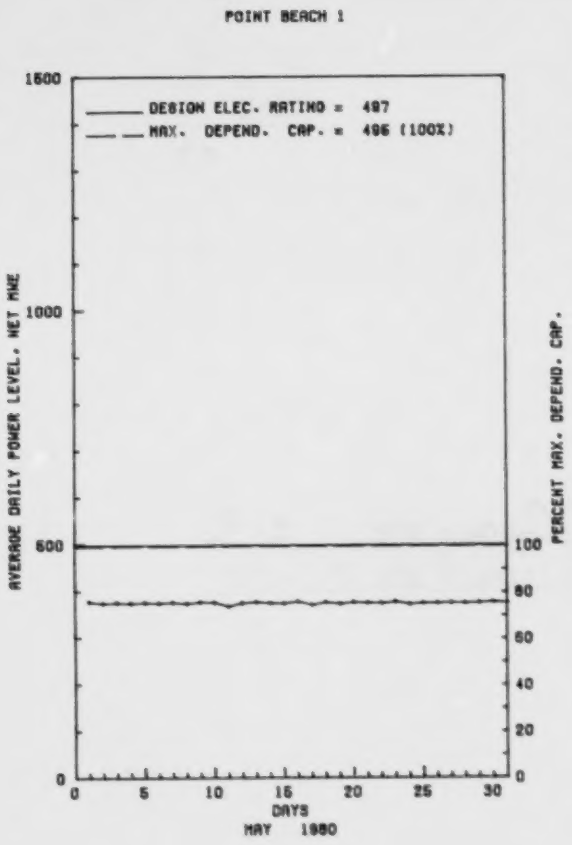
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-04/ 03L-0	04/09/80	04/25/80	Emergency Diesel Generator failed to start.
80-06/ 03L-0	04/17/80	05/05/80	An overstressed condition in the component cooling system piping.

REVIEWED BY OMPA LOEB: T. C. Cintula **DATE:** 7 · 3 · 80

REVIEWED BY NRR DOR: C M Tramell **DATE:** 7-7-80

POINT BEACH 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-266
2. Utility Contact: C.W. FAY (414) 277-2811
3. Licensed Thermal Power (MWt): 1518
4. Nameplate Rating (Gross MWe): 582 x 0.9 = 524
5. Design Electrical Rating (Net MWe): 497
6. Maximum Dependable Capacity (Gross MWe): 519
7. Maximum Dependable Capacity (Net MWe): 495
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): 390
10. Reasons for Restrictions, If Any:
INHIBIT CORROSION OF STEAM GENERATOR TUBES.

ITEM 21 CALCULATED BY USING A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	3,647.0	83,879.0
12. Hours Reactor Was Critical	744.0	2,898.5	69,415.0
13. Reactor Reserve Shutdown Hours	.0	18.8	601.0
14. Hours Generator On-Line	744.0	2,755.5	67,149.5
15. Unit Reserve Shutdown Hours	.0	151.5	753.9
16. Gross Thermal Energy (MWH)	981,815	3,314,926	94,867,719
17. Gross Electrical Energy (MWH)	294,140	1,081,680	31,715,430
18. Net Electrical Energy (MWH)	278,216	1,021,793	30,211,971
19. Unit Service Factor	100.0	75.6	80.1
20. Unit Availability Factor	100.0	79.7	81.0
21. Unit Capacity Factor (MDC Net)	75.5	56.6	74.0
22. Unit Capacity Factor (DER Net)	75.2	56.4	72.5
23. Unit Forced Outage Rate	.0	.0	3.6

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
SCHED SHUTDOWN 7/25/80 FOR GEN LEAK TEST & EDD CURRENT INSP.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
NONE									

SUMMARY

THE UNIT OPERATED AT 375 MWE DURING THE PERIOD WITH ONE LOAD REDUCTION ON 05/11/80. THE REDUCTION WAS FOR TURBINE STOP VALVE TESTING.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

POINT BEACH 2

FACILITY DATA

Facility Description

1. Location (state/county) (nearest population center)	Wisconsin/Manitowoc 15 miles N of Manitowoc
2. Electric Reliability Council	Mid-America Interpool Network
3. Type of Reactor	PWR
4. Power Level (MWT)	1518
5. Design Electrical Rating (MWE Net)	497
6. Date of Initial Criticality	May 30, 1972
7. Date of Commercial Operation	October 1, 1972
8. Condenser Cooling Method	Straight Through
9 Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10. Licensee	Wisconsin Electric Power Co.
11. Corporate Address	231 West Michigan Milwaukee, Wisconsin
12. Corporate Contact	S. Burstein
13. Architect/Engineer	Bechtel
14. Nuclear Steam Supply System	Westinghouse
15. Constructor	Bechtel

Regulatory Information

16. IE Region responsible	III
17. IE Branch Chief	R. Heishman
18. IE Section Chief	R. F. Warnick
19. IE Resident Inspector	J. D. SMITH
20. Licensing Project Manager	C. M. Trammell
21. Docket Number	50-301
22. License Number/Date of Issue	DPR-27 issued 11/18/71
23. Public Document Room	Document Department, Library University of Wisconsin-Stevens Point Stevens Point, Wisconsin 54481 Manitowoc Public Library Manitowoc, Wisconsin (Temporary LPDR for spent fuel pool capacity increase amendment only

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

The unit was put online May 14, 1980 following outage for refueling and maintenance.

LAST IE SITE INSPECTION DATE: March 10-14 and April 8, 1980

INSPECTION REPORT NO: 80-08

SUMMARY

INSPECTION SUMMARY

Inspection on March 31 through April 4, (80-05): Included a review of the Security Plan; Security Program Audit; Security Organization; Protected Area Physical Barriers; Vital Area Physical Barriers; Lighting; Access Control (Identification, Authorization and Badging); Access Control (Search); Access Control (Escorting); Access Control (Vital Areas); Alarm Stations; Detection Aids; Assessment Aids; Communications; Testing and Maintenance; Compensatory Measures; Power Supply; Response; Locks, Keys and Combinations; Records and Reports; Licensee Action of Previous Inspection Findings; Independent Inspection Effort. The inspection involved 72 inspector hours onsite by two NRC inspectors. The inspection began during the day shift; 19 inspection hours were accomplished during offshift periods. The licensee was found to be in compliance with NRC requirements within the 23 areas examined during the inspection except for the following 3 items: Vital Area Physical Barriers - Some vital area physical barriers were not properly maintained; Alarm Stations - A change in alarm status was not properly monitored; and Testing and Maintenance - Some alarm tests were not done at the required frequency.

Inspection on April 8-10, (80-06): Routine, unannounced inspection of radiation protection program associated with a refueling outage, including: radiation protection procedures; advanced planning and preparation; training; exposure control; instruments and equipment; posting and control; IE Information Notice No. 79-08; IE Circular No. 79-15; contract radiation protection technician qualifications; and specific matters concerning radioactive materials transport. The inspection involved 36 inspector hours onsite by two NRC inspectors. No items of noncompliance or deviations were identified.

Inspection on March 10-14 and April 8, (80-08): Routine, announced inspection on March 10-14, 1980 to review plant procedures, operator training, and systems utilized for Small Break Loss of Coolant event and other events affecting primary coolant inventory. On April 8, 1980 a special announced management meeting between NRC staff members, corporation management representatives of Wisconsin Electric Power, and Point Beach plant management to review plant performance was held. The inspection involved 55 inspector hours onsite by two NRC inspectors. The meeting involved 15 manhours by five NRC representatives. No items of noncompliance or deviations were identified.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-04/ 03L-0	04/17/80	05/05/80	Overstressed condition in the component system piping.

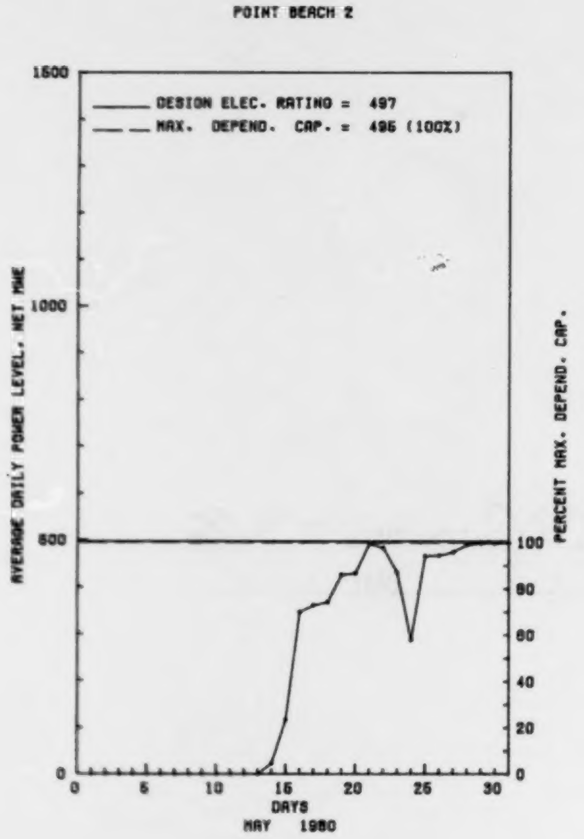
REVIEWED BY OMPA LOEB: T. C. Cintula **DATE:** 7. 3. 80

REVIEWED BY NRR DOR: CM Tramell **DATE:** 7-7-80

GPO 871-087

POINT BEACH 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-301
2. Utility Contact: C.W. FAY (414) 277-2811
3. Licensed Thermal Power (MWt): 1518
4. Nameplate Rating (Gross MWe): 582 x 0.9 = 524
5. Design Electrical Rating (Net MWe): 497
6. Maximum Dependable Capacity (Gross MWe): 519
7. Maximum Dependable Capacity (Net MWe): 495
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

ITEM 21 CALCULATED BY USING A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	3,647.0	68,669.0
12. Hours Reactor Was Critical	458.7	2,546.9	61,443.8
13. Reactor Reserve Shutdown Hours	9.8	19.2	185.3
14. Hours Generator On-Line	407.7	2,480.7	60,207.7
15. Unit Reserve Shutdown Hours	10.4	13.5	119.9
16. Gross Thermal Energy (MWH)	539,505	3,566,569	81,219,105
17. Gross Electrical Energy (MWH)	182,120	1,186,790	27,548,130
18. Net Electrical Energy (MWH)	170,733	1,127,626	26,202,753
19. Unit Service Factor	54.8	68.0	87.7
20. Unit Availability Factor	56.2	68.4	87.9
21. Unit Capacity Factor (MDC Net)	46.4	62.5	77.8
22. Unit Capacity Factor (DER Net)	46.2	62.2	76.8
23. Unit Forced Outage Rate	.0	12.4	2.0

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
2	04/11/80	S	326.6	C	4		HA	TURBIN	INSPECTION OF TURBINE ROTORS FOR INDICATIONS.
3	05/15/80	S	2.6	B	1		HA	TURBIN	TURBINE OVERSPEED TRIP TESTING.
4	05/23/80	S	7.0	B	1		HB	VALVEX	MOISTURE SEPARATOR REHEATER DUMP VALVES WERE CLOGGED CAUSING LEVEL CONTROL PROBLEMS. VALVES WERE CLEANED DURING THE SHUTDOWN.

SUMMARY

THE UNIT RETURNED TO SERVICE ON 05/14/80 AND OPERATED NEAR FULL POWER EXCEPT FOR A TURBINE TRIP OVERSPEED TEST ON 05/15/80 AND A SHUTDOWN FOR VALVE CLEANING ON 05/23/80.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

PRAIRIE ISLAND 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Minnesota/Goodhue 28 miles SE of Minneapolis
2.	Electric Reliability Council	Mid-Continent Area Reliability Coordination Agreement
3.	Type of Reactor	PWR
4.	Power Level (MWT)	1650
5.	Design Electrical Rating (MWE Net)	530
6.	Date of Initial Criticality	December 1, 1973
7.	Date of Commercial Operation	December 16, 1973
8.	Condenser Cooling Method	Cooling Towers
9.	Condenser Cooling Water Source	Mississippi River

Utility & Contractor Information

10.	Licensee	Northern States Power Company
11.	Corporate Address	414 Nicollet Mall - 8th Floor Minneapolis, Minnesota 55401
12.	Corporate Contact	G. H. Neils (Oper.) L. O. Mayer (Lic.)
13.	Architect/Engineer	Pioneer Services & Eng. Co.
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Northern States Power Co.

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	D. C. BOYD
19.	IE Resident Inspector	C. D. Feierabend 612-388-8209
20.	Licensing Project Manager	R. Martin
21.	Docket Number	50-282
22.	License Number/Date of Issue	DPR-42 issued 8/9/73
23.	Public Document Room	Environmental Conservation Library Minneapolis Public Library 300 Nicollet Mall Minneapolis, Minnesota 55401

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

Operating routinely.

LAST IE SITE INSPECTION DATE: April 28 through May 2, 1980

INSPECTION REPORT NO: 80-07

SUMMARY

INSPECTION SUMMARY

Inspection on April 1-30, (80-06): Routine resident inspection of plant operation, maintenance, surveillance, security, radiation protection, followup of licensee reported events, followup of IE Bulletin and independent verifications of operability of safety-related systems. The inspection involved 105 inspector hours onsite by one NRC inspector. No items of noncompliance were identified. Of the eight areas inspected, no deviations were identified in seven areas. One deviation from a commitment was identified in one area (failure to complete inspections required by IE Bulletin 78-05).

Inspection on April 28 through May 2, (80-07): Included a review of the Security Plan; Security Program Audit; Security Organization; Protected Area Physical Barriers; Vital Area Physical Barriers; Lighting; Access Control (Identification, Authorization and Badging); Access Control (Search); Access Control (Escorting); Access Control (Vital Areas); Alarm Stations; Detection Aids; Assessment Aids; Communications; Testing and Maintenance; Compensatory Measures; Power Supply; Response; Locks, Keys and Combinations; Records and Reports; Licensee Action on Previous Inspection Findings; Independent Inspection Effort. The inspection involved 75 inspector hours onsite by two NRC inspectors including 14 inspection hours accomplished during offshift periods. The inspection began during the dayshift. The licensee was found to be in compliance with NRC requirements within the 20 areas examined during the inspection except for the following 5 items: Security Organization - The corrected vision of some security force personnel did not meet minimum requirements identified in the approved security plan; Vital Area Physical Barriers - Some vital area physical barriers were not properly maintained; Access Control (Escorting) - An unattended designated vehicle located within the protected area was not disabled as required by the approved security plan; Detection Aids - The instrusion detection system (E-Field) was not adequately employed to detect penetration at some location of the protected area boundary; and Detection Aids - Some vital area alarm devices and monitoring systems were not tamper-indicating.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
79-29/03L-0	11/02/79	11/79	Turbine driven feedwater pump did not start for surveillance test.

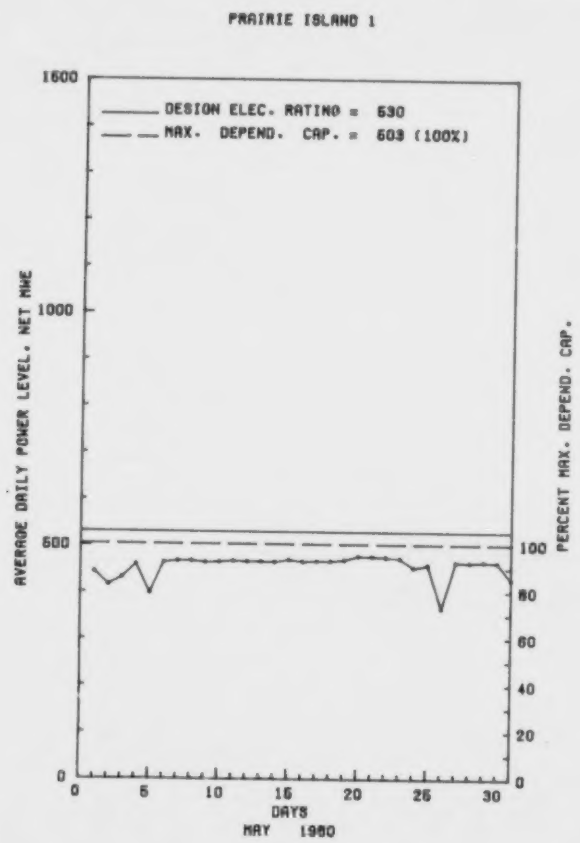
==

REVIEWED BY OMPA LOEB: T. C. Cintula **DATE:** 7. 3. 80

REVIEWED BY NRR DOR: RS Martin **DATE:** 7/7/80

PRAIRIE ISLAND 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-282
2. Utility Contact: A. A. Hunstad (612) 388-4767
3. Licensed Thermal Power (MWt): 1650
4. Nameplate Rating (Gross MWe): 659 x 0.9 = 593
5. Design Electrical Rating (Net MWe): 530
6. Maximum Dependable Capacity (Gross MWe): 534
7. Maximum Dependable Capacity (Net MWe): 503
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	3,647.0	56,615.0
12. Hours Reactor Was Critical	744.0	3,515.9	44,724.3
13. Reactor Reserve Shutdown Hours	.0	.0	5,531.9
14. Hours Generator On-Line	744.0	3,510.8	43,634.4
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,201,936	5,706,058	67,785,052
17. Gross Electrical Energy (MWH)	362,800	1,773,480	21,927,150
18. Net Electrical Energy (MWH)	339,781	1,663,383	20,502,635
19. Unit Service Factor	100.0	96.3	77.1
20. Unit Availability Factor	100.0	96.3	77.1
21. Unit Capacity Factor (MDC Net)	90.8	90.7	72.0
22. Unit Capacity Factor (DER Net)	86.2	86.1	68.3
23. Unit Forced Outage Rate	.0	.0	12.6

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
REFUELING, AUGUST 4, 1980, 8 WEEKS
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	NONE								

SUMMARY

THE UNIT WAS IN LOAD FOLLOWING MODE FOR THIS MONTH. IT'S POWER LEVELS WERE LIMITED FOR THE FIRST FEW DAYS OF THE MONTH DUE TO LICENSE LIMITATION ON RIVER DELTA T.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same Source.

PRAIRIE ISLAND 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1. Location (state/county) (nearest population center) — Minnesota/Goodhue, 28 miles SE of Minneapolis
2. Electric Reliability Council — Mid-Continent Area Reliability Coordination Agreement
3. Type of Reactor — PWR
4. Power Level (MWT) — 1650
5. Design Electrical Rating (MWE Net) — 530
6. Date of Initial Criticality — December 17, 1974
7. Date of Commercial Operation — December 21, 1974
8. Condenser Cooling Method — Cooling Towers
9. Condenser Cooling Water Source — Mississippi River

Utility & Contractor Information

10. Licensee — Northern States Power Company
11. Corporate Address — 414 Nicollet Mall - 8th Floor, Minneapolis, Minnesota 55401
12. Corporate Contact — G. H. Neils (Oper.), L. O. Mayer (Lic.)
13. Architect/Engineer — Pioneer Services & Eng. Co.
14. Nuclear Steam Supply System — Westinghouse
15. Constructor — Northern States Power Co.

Regulatory Information

16. IE Region responsible — III
17. IE Branch Chief — R. Heishman
18. IE Section Chief — D. C. BOYD
19. IE Resident Inspector — C. D. Feierabend 612-388-8209
20. Licensing Project Manager — R. Martin
21. Docket Number — 50-306
22. License Number/Date of Issue — DPR-60 issued 10/29/74
23. Public Document Room — Environmental Conservation Library, Minneapolis Public Library, 300 Nicollet Mall, Minneapolis, Minnesota 55401

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

Operating routinely.

LAST IE SITE INSPECTION DATE: April 28 through May 2, 1980

INSPECTION REPORT NO: 80-08

SUMMARY

INSPECTION SUMMARY

Inspection on April 1-30, (80-07): Routine resident inspection of plant operation, maintenance, surveillance, security, radiation protection, followup of licensee reported events, followup of IE Bulletin and independent verifications of operability of safety-related systems. The inspection involved 105 inspector hours onsite by one NRC inspector. No items of noncompliance were identified. Of the eight areas inspected, no deviations were identified in seven areas. One deviation from a commitment was identified in one area (failure to complete inspections required by IE Bulletin 78-05).

Inspection on April 28 through May 2, (80-08): Included a review of the Security Plan; Security Program Audit; Security Organization; Protected Area Physical Barriers; Vital Area Physical Barriers; Lighting; Access Control (Identification, Authorization and Badging); Access Control (Search); Access Control (Escorting); Access Control (Vital Areas); Alarm Stations; Detection Aids; Assessment Aids; Communications; Testing and Maintenance; Compensatory Measures; Power Supply; Response; Locks, Keys and Combinations; Records and Reports; Licensee Action on Previous Inspection Findings; Independent Inspection Effort. The inspection involved 75 inspector hours onsite by two NRC inspectors including 14 inspection hours accomplished during offshift periods. The inspection began during the dayshift. The licensee was found to be in compliance with NRC requirements within the 20 areas examined during the inspection except for the following 5 items: Security Organization - The corrected vision of some security force personnel did not meet minimum requirements identified in the approved security plan; Vital Area Physical Barriers - Some vital area physical barriers were not properly maintained; Access Control (Escorting) - An unattended designated vehicle located within the protected area was not disabled as required by the approved security plan; Detection Aids - The instrusion detection system (E-Field) was not adequately employed to detect penetration at some location of the protected area boundary; and Detection Aids - Some vital area alarm devices and monitoring systems were not tamper-indicating.

REPORTS RECEIVED FROM LICENSEE

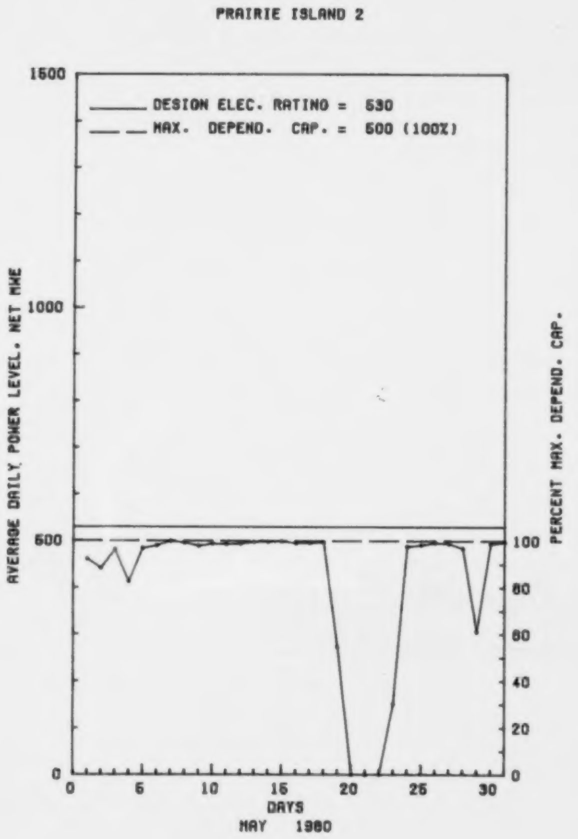
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-11/ 03L-0	03/12/80	04/11/80	With one charging pump out of service No. 22 charging pump was started, but its motor breaker would not stay closed and the pump stopped.
80-13/ 03L-0	04/11/80	05/09/80	While performing maintenance in the area, a workman accidentally tripped the overspeed trip valve for No. 22 Turbine-Driven Aux Feedwater Pump.
80-14/ 03L-0	04/10/80	05/09/80	During surveillance test, one overpower detla T setpoint was found to trip about 70% above the desired setpoint.

REVIEWED BY OMPA LOEB: T. C. Cintula **DATE:** 7-3-80

REVIEWED BY NRR DOR: R. Martin **DATE:** 7/7/80

PRAIRIE ISLAND 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-306
2. Utility Contact: A. A. Hunstad (612) 388-4767
3. Licensed Thermal Power (MWt): 1650
4. Nameplate Rating (Gross MWe): .659 x 0.9 = 593
5. Design Electrical Rating (Net MWe): 530
6. Maximum Dependable Capacity (Gross MWe): 531
7. Maximum Dependable Capacity (Net MWe): 500
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	3,647.0	47,733.0
12. Hours Reactor Was Critical	653.2	2,385.1	41,516.1
13. Reactor Reserve Shutdown Hours	.0	42.4	1,448.4
14. Hours Generator On-Line	646.2	2,330.7	40,731.7
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,038,769	3,678,667	63,351,170
17. Gross Electrical Energy (MWH)	329,920	1,190,848	20,244,970
18. Net Electrical Energy (MWH)	308,831	1,111,227	18,949,859
19. Unit Service Factor	86.9	63.9	85.3
20. Unit Availability Factor	86.9	63.9	85.3
21. Unit Capacity Factor (MDC Net)	83.0	60.9	79.4
22. Unit Capacity Factor (DER Net)	78.3	57.5	74.9
23. Unit Forced Outage Rate	.3	.5	4.6

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUELING, FEB. 1981, 6 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	05/19/80	S	95.8	B	1		CB	PUMPXX	SHUTDOWN TO REPAIR NO. 2 SEAL ON NO. 22 RCP.
	05/29/80	F	2.0	B	1		EG	TRANSF	SHUTDOWN TO REMOVE 2M TRANSFORMER FROM SERVICE.

SUMMARY

THE UNIT WAS IN LOAD FOLLOWING MODE FOR THE MONTH. POWER WAS LIMITED FOR THE FIRST FEW DAYS DUE TO LICENSE LIMITATION ON RIVER DELTA T. ON 05/19/80, AN OUTAGE WAS TAKEN TO REPAIR A SEAL ON A COOLANT PUMP. ON 05/24/80, ANOTHER OUTAGE WAS TAKEN TO REMOVE A TRANSFORMER FROM SERVICE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

QUAD CITIES 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Illinois/Rock Island 20 miles NE of Moline
2.	Electric Reliability Council	Mid-America Interpool Network
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2511
5.	Design Electrical Rating (MWE Net)	789
6.	Date of Initial Criticality	October 18, 1971
7.	Date of Commercial Operation	February 18, 1973
8.	Condenser Cooling Method	Spray Canal
9.	Condenser Cooling Water Source	Mississippi River

Utility & Contractor Information

10.	Licensee	Commonwealth Edison Co.
11.	Corporate Address	P. O. Box 767 Chicago, Illinois 60690
12.	Corporate Contact	D. L. Peoples
13.	Architect/Engineer	Sargent and Lundy
14.	Nuclear Steam Supply System	GE
15.	Constructor	United Engineers & Constructors, Inc.

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	R. Spessard
19.	IE Resident Inspector	N. J. Chrissotimus (309) 654-2227
20.	Licensing Project Manager	R. B. Bevan
21.	Docket Number	50-254
22.	License Number/Date of Issue	DPR-29 issued 10/1/71
23.	Public Document Room	Moline Public Library 504 17th Street Moline, Illinois 61265

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

Unit in coastdown mode at end of operating cycle. Refueling outage scheduled for September 1, 1980.

LAST IE SITE INSPECTION DATE: March 27 through May 7, 1980

INSPECTION REPORT NO: 80-10

SUMMARY

INSPECTION SUMMARY

Inspection on April 1-2, (80-09): Routine, unannounced inspection of initial training and retraining of licensed and non-licensed personnel, and licensed operator requalification training. The inspection involved 13 inspector hours onsite by one NRC inspector. Of the areas inspected, no items of noncompliance were identified.

Inspection on March 27 through May 7, (80-10): Routine, unannounced inspection consisting of operational safety verification, long-term shut-down activities, maintenance observation (annual and refuel), surveillance observation, licensee event report followup, IE Bulletin followup, IE Circular followup, startup testing-refueling, plant trips, and independent inspection. The inspection involved 195 inspector hours onsite by one NRC inspector. No items of noncompliance were identified.

REPORTS RECEIVED FROM LICENSEE

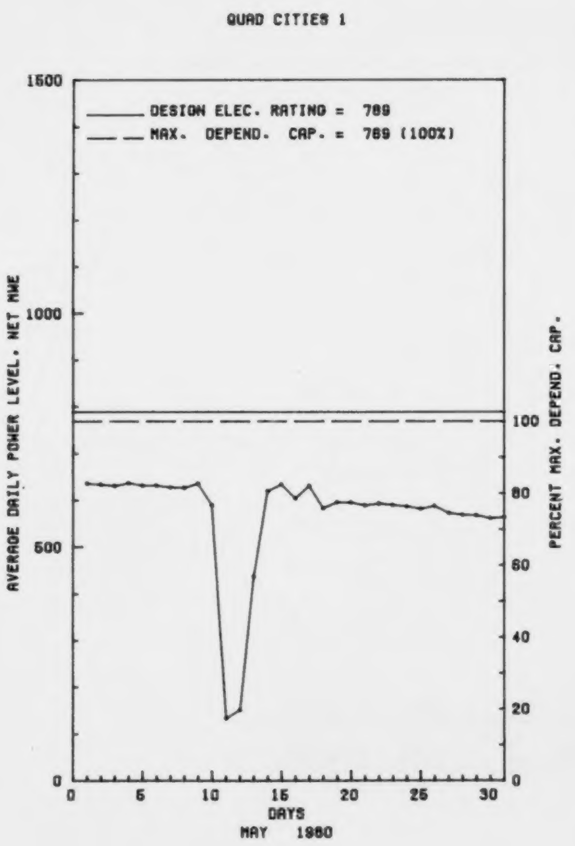
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-09/ 03L-0	04/10/80	04/30/80	During test procedure QOS 1000-3, the thermal overload relay for the B-loop suppression chamber cooling valve tripped while attempting to open.
80-10/ 03L-0	04/23/80	05/16/80	During Electrohydraulic Control Fluid Pressure Scram surveillance, prs. switch 1-5600-PS-3 tripped at 896 psig.
80-13/ 03L-0	04/15/80	05/14/80	During procedure QIS 10-1, swithc LIS-1-263-57A tripped at a level of 3.85 below TS 3.1.A LCO.
80-14/ 03L-0	04/15/80	05/14/80	During procedure QIS-6, pressure switch PS-1-1001-88A was found to trip at 2.05 psig.

REVIEWED BY OMPA LOEB: John J. Rizzo DATE: 7-3-80

REVIEWED BY NRR DOR: RBBevan DATE: 7/8/80

QUAD CITIES 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-254
2. Utility Contact: D. Hannum (309) 654-2241
3. Licensed Thermal Power (MWt): 2511
4. Nameplate Rating (Gross MWe): 920 x 0.9 = 828
5. Design Electrical Rating (Net MWe): 789
6. Maximum Dependable Capacity (Gross MWe): 813
7. Maximum Dependable Capacity (Net MWe): 769
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	3,647.0	70,687.0
12. Hours Reactor Was Critical	744.0	3,647.0	58,361.1
13. Reactor Reserve Shutdown Hours	.0	.0	3,421.9
14. Hours Generator On-Line	731.8	3,634.8	55,675.7
15. Unit Reserve Shutdown Hours	.0	.0	909.2
16. Gross Thermal Energy (MWH)	1,456,691	8,098,710	112,642,691
17. Gross Electrical Energy (MWH)	465,302	2,638,383	36,261,437
18. Net Electrical Energy (MWH)	423,479	2,474,620	33,890,156
19. Unit Service Factor	98.4	99.7	78.9
20. Unit Availability Factor	98.4	99.7	80.1
21. Unit Capacity Factor (MDC Net)	74.0	88.2	62.4
22. Unit Capacity Factor (DER Net)	72.1	86.0	60.8
23. Unit Forced Outage Rate	1.6	.3	7.2

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUELING, 09/01/80
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
7	05/11/80	F	12.2	A	1		CB	CKTBRK	REPLACE 1B RECIRC M-G SET FIELD BREAKER TRIP COIL; REACTOR WAS PLACED IN HOT STANDBY.

SUMMARY

THE UNIT OPERATED ROUTINELY EXCEPT FOR A SHUTDOWN ON 05/11/80 FOR A RECIRC MOTOR-GENERATOR BREAKER REPLACEMENT. LOAD INCREASE WAS DELAYED TWICE FOR MONITOR CALIBRATION AND DEMINERALIZER REPLACEMENT.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

QUAD CITIES 2

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Illinois/Rock Island 20 miles NE of Moline
2.	Electric Reliability Council	Mid-America Interpool Network
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2511
5.	Design Electrical Rating (MWE Net)	789
6.	Date of Initial Criticality	April 26, 1972
7.	Date of Commercial Operation	March 10, 1973
8.	Condenser Cooling Method	Spray Canal
9.	Condenser Cooling Water Source	Mississippi River

Utility & Contractor Information

10.	Licensee	Commonwealth Edison Co.
11.	Corporate Address	P. O. Box 767 Chicago, Illinois 60690
12.	Corporate Contact	D. L. Peoples
13.	Architect/Engineer	Sargent & Lundy
14.	Nuclear Steam Supply System	GE
15.	Constructor	United Engineers & Constructors, Inc.

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	R. Spessard
19.	IE Resident Inspector	N. J. Chrissotimus (309) 654-2227
20.	Licensing Project Manager	R. B. Bevan
21.	Docket Number	50-265
22.	License Number/Date of Issue	DPR-30 issued 3/21/72
23.	Public Document Room	Moline Public Library 504 17th Street Moline, Illinois 61265

INSPECTION STATUS

ENFORCEMENT STATUS

TECHNICAL SPECIFICATION 6.2.A REQUIRES THAT "DETAILED WRITTEN PROCEDURES INCLUDING APPLICABLE CHECKOFF LISTS...SHALL BE PREPARED, APPROVED, AND ADHERED TO." CONTRARY TO THE ABOVE, PROCEDURE QTS 150-57, REVISION 6, "UNIT 2 IPCLRT VALVE LINEUP," WAS NOT ADHERED TO IN THAT VALVE POSITION VERIFICATION WAS NOT CARRIED OUT ON MARCH 8, 1980, TO VERIFY THAT VALVES 2-1402-33A AND 2-1402-33B (LOCAL LEAK TEST VALVES FOR CORE SPRAY ISOLATION VALVES) WERE LOCKED CLOSED AS REQUIRED. (8001 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

FACILITY ITEMS

Unit 2 was returned to operation on April 23, 1980 following a 21 week refuel/maintenance outage. Major work included feedwater nozzle clad removal, Mark I containment modifications, repair of core spray piping and replacement of a jet pump holddown beam assembly (80-13). Routine operations in the month of May.

PLANT STATUS

Plant operating routinely.

LAST IE SITE INSPECTION DATE: March 27 through May 7, 1980

INSPECTION REPORT NO: 80-13

SUMMARY

INSPECTION SUMMARY

Inspection on March 12-13, 26-28; April 3-4 and 11, (80-11): Review of Core Spray System repair program and repair procedures, observation of repair welding activities and NDE examinations, review of personnel and material certifications, and review of repair documentation and radiographs. The inspection involved 47 inspector hours onsite by one NRC inspector. No items of noncompliance or deviations were identified.

Inspection on April 1-2, (80-12): Routine, unannounced inspection of initial training and retraining of licensed and non-licensed personnel, and licensed operator requalification training. The inspection involved 13 inspector hours onsite by one NRC inspector. Of the areas inspected, no items of noncompliance were identified.

Inspection on March 27 through May 7, (80-13): Routine, unannounced inspection consisting of operational safety verification, long-term shutdown activities, maintenance observation (annual and refuel), surveillance observation, licensee event report followup, IE Bulletin followup, IE Circular followup, startup testing-refueling, plant trips, and independent inspection. The inspection involved 195 inspector hours onsite by one NRC inspector. No items of noncompliance were identified.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

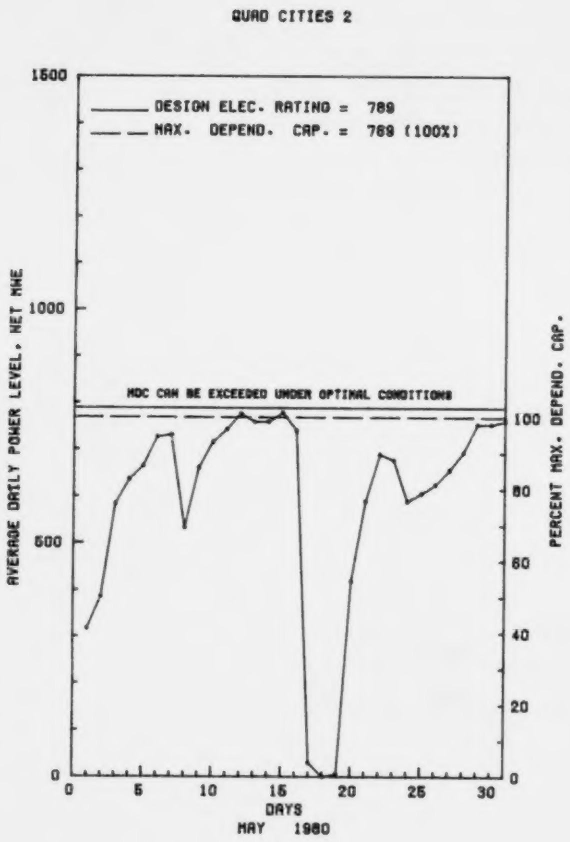
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-03/ 03L-0	02/13/80	03/01/80	Reactor building ventilation would not reset following a test.
80-09/ 03L-0	03/22/80	04/21/80	Proc. QOS 1600-13, TIP machine No. 2 failed to auto. withdraw the TIP when Gr. II ispaltion signal was initiated.
80-11/ 03L-0	04/20/80	05/16/80	While performing the Manual Operation of Relief Valves, procedure QOS 201-1, relief valve 2-203-3C failed to open.

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 7-3-80

REVIEWED BY NRR DOR: RBBevan **DATE:** 7/8/80

QUAD CITIES 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-265
2. Utility Contact: D. Mannum (309) 654-2241
3. Licensed Thermal Power (MWt): 2511
4. Nameplate Rating (Gross MWe): 920 x 0.9 = 828
5. Design Electrical Rating (Net MWe): 789
6. Maximum Dependable Capacity (Gross MWe): 813
7. Maximum Dependable Capacity (Net MWe): 769
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	3,647.0	69,717.0
12. Hours Reactor Was Critical	696.1	873.4	54,002.3
13. Reactor Reserve Shutdown Hours	.0	.0	2,985.8
14. Hours Generator On-Line	664.5	784.2	51,576.0
15. Unit Reserve Shutdown Hours	.0	.0	702.9
16. Gross Thermal Energy (MWH)	1,448,078	1,615,422	104,631,531
17. Gross Electrical Energy (MWH)	462,506	509,491	33,372,541
18. Net Electrical Energy (MWH)	440,847	442,575	31,185,100
19. Unit Service Factor	89.3	21.5	74.0
20. Unit Availability Factor	89.3	21.5	75.0
21. Unit Capacity Factor (MDC Net)	77.1	15.8	58.2
22. Unit Capacity Factor (DER Net)	75.1	15.4	56.7
23. Unit Forced Outage Rate	10.7	20.1	9.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
3	05/01/80	F	12.9	A	3		CH	VALVEX	REACTOR SCRAM FROM LOW VESSEL WATER LEVEL DUE TO 2B FEEDWATER REGULATION VALVE FAILED CLOSED.
4	05/04/80	F	.0	A	5				LOAD REDUCTION DUE TO FALSE SENSOR READING ON HIGH PRESSURE TURBINE VIBRATION.
5	05/07/80	F	.0	A	5				LOAD REDUCTION DUE TO FEEDWATER FLOW REDUCTION TEST AND DRYWELL ENTRY TO REPAIR 2B RECIRC PUMP OIL LEVEL INDICATION.
6	05/17/80	F	66.6	B	1		HH	XXXXXX	SHUTDOWN TO REPAIR MAIN CONDENSER TUBE LEAKS.

SUMMARY

THE UNIT OPERATED ROUTINELY EXCEPT FOR SHUTDOWNS ON 05/01/30 AND ON 05/17/80 DUE TO LOW WATER LEVEL AND FOR CONDENSER TUBE REPAIR AND FOR REDUCTIONS ON 5/4 AND 5/7.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

RANCHO SECO

NRC POWER RESTRICTIONS: None

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	California/Sacramento 25 miles SE of Sacramento
2.	Electric Reliability Council	Western Systems Coordinating Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2772
5.	Design Electrical Rating (MWE Net)	918
6.	Date of Initial Criticality	September 16, 1974
7.	Date of Commercial Operation	April 17, 1975 (92.6% of full power)
8.	Condenser Cooling Method	Closed Cycle
9.	Condenser Cooling Water Source	Natural Draft Cooling Towers - Folsom Canal

Utility & Contractor Information

10.	Licensee	Sacramento Municipal Utility District
11.	Corporate Address	6201 South Street P. O. Box 15830 Sacramento, California
12.	Corporate Contact	J. J. Mattimoe Assistant General Manager, Chief Engineer
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	B&W
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	V
17.	IE Branch Chief	J. L. Crews
18.	IE Section Chief	B. H. Faulkenberry
19.	IE Resident Inspector	H. L. Canter (209) 748-2791
20.	Licensing Project Manager	D. J. Garner
21.	Docket Number	50-312
22.	License Number/Date of Issue	DPR-54, 8/16/74
23.	Public Document Room	Business and Municipal Department Sacramento City-County Library 828 I Street Sacramento, California 95814

INSPECTION STATUS

ENFORCEMENT STATUS

"TECHNICAL SPECIFICATION 6.5.1.6.A READS, IN PART, "THE PLANT REVIEW COMMITTEE (PRC) SHALL BE RESPONSIBLE FOR REVIEW OF (1) ALL PROCEDURES REQUIRED BY SPECIFICATION 6.8 AND CHANGES THERETO...." CONTRARY TO THESE REQUIREMENTS, THE PROCEDURES REQUIRED FOR CONTROLLING CERTAIN USES OF THE POLAR CRANE HAD NOT BEEN REVIEWED BY THE PRC PRIOR TO FEBRUARY 19, 1980, WHEN THE PROCEDURES WERE USED TO MOVE A LARGE 3,000 POUND CALIBRATION BLOCK OVER THE FUEL TRANSFER CANAL WITH THE REACTOR VESSEL HEAD REMOVED AND IRRADIATED FUEL IN THE REACTOR BUILDING. (8010 INFRACTION)

INSPECTION STATUS

SYSTEMS AND COMPONENT PROBLEMS:

None

FACILITY ITEMS (PLANS AND PROCEDURES):

None.

MANAGERIAL ITEMS:

Awaiting decision of ASLB.

PLANT STATUS:

Operation at or near full power

LAST IE SITE INSPECTION DATE: 05/05-30/80+

INSPECTION REPORT NO. 50-312/80-17+

SUMMARY

INSPECTION SUMMARY

Inspection on May 5-9, 1980 (Report No. 50-312/80-13)
Report being prepared; to be reported next month.

Inspection on April 14-26, 1980 (Report No. 50-312/80-15)
Report being prepared; to be reported next month.

Inspection on April 1-30, 1980 (Report No. 50-312/80-16)
Areas Inspected: Routine inspections of long term shutdown activities; followup on a Regional request item; and independent inspection effort. The inspection involved 106 inspector hours by the Senior Resident inspector and 54 inspector hours by other NRC inspectors.

Results: No items of noncompliance or deviations were identified.

Inspection on May 1-30, 1980 (Report No. 50-312/80-17)
Report being prepared; to be reported next month.

REPORTS RECEIVED FROM LICENSEE

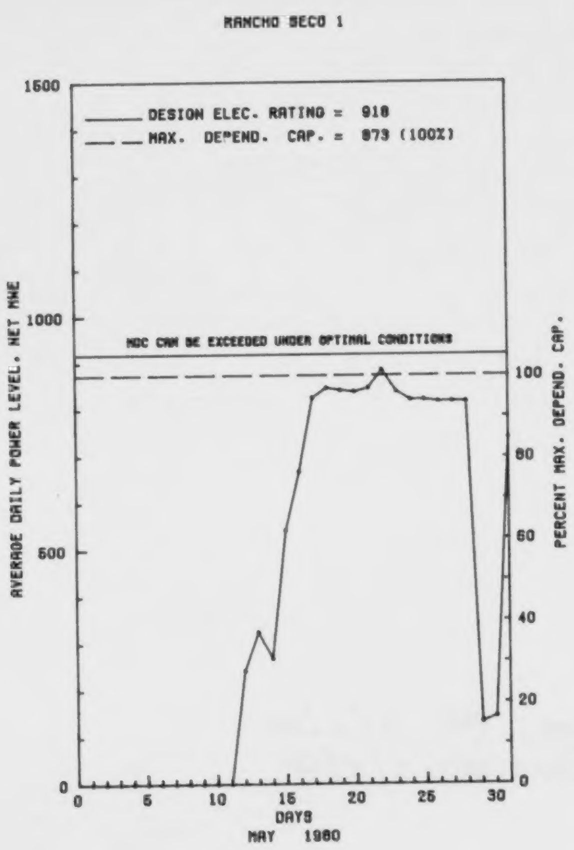
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-19/ 03L-0	04-06-80	05-05-80	ITE 5 KV type 5HK350 bkr failed to open due to dirt and hard lube.
80-21/ 04T-0	03-18-80	04-21-80	Effluent PH exceeded TS limit of 8.5 for about three hours.
80-22/ 03L-0	04-06-80	04-29-80	Bad contacts on BLPB cause misoperation of cont isolation valve.
80-24/ 03L-0	04-09-80	04-30-80	DSL generator A failed to reach voltage in 10 seconds due to missing coupling.
80-25/ 03L-0	05-02-80	05-20-80	Per Information Notice 80-13, found several GE type SBM switches.
80-26/ 01T-0	05-08-80	05-08-80	Subcooling requirement results in greater offsite dose for OTSG leak.
80-27/ 03L-0	05-08-80	05-20-80	BP snubber 45W-25023-5A low on fluid.

REVIEWED BY OMPA LOEB: PE Bole DATE: 7/3/80

REVIEWED BY NRR DOR: D J Garner DATE: 7/7/80

RANCHO SECO

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-312
2. Utility Contact: R. W. Colombo (916) 452-3211
3. Licensed Thermal Power (MWt): 2772
4. Nameplate Rating (Gross MWe): 1070 x 0.9 = 963
5. Design Electrical Rating (Net MWe): 918
6. Maximum Dependable Capacity (Gross MWe): 917
7. Maximum Dependable Capacity (Net MWe): 873
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	3,647.0	44,904.0
12. Hours Reactor Was Critical	523.0	794.0	26,867.5
13. Reactor Reserve Shutdown Hours	.0	.0	3,975.1
14. Hours Generator On-Line	442.7	709.6	25,650.9
15. Unit Reserve Shutdown Hours	.0	.0	1,210.2
16. Gross Thermal Energy (MWH)	995,231	1,685,051	64,718,749
17. Gross Electrical Energy (MWH)	336,024	557,898	21,842,344
18. Net Electrical Energy (MWH)	318,424	516,866	20,627,756
19. Unit Service Factor	59.5	19.5	57.1
20. Unit Availability Factor	59.5	19.5	59.8
21. Unit Capacity Factor (MDC Net)	49.0	16.2	52.6
22. Unit Capacity Factor (DER Net)	46.6	15.4	50.0
23. Unit Forced Outage Rate	7.3	10.1	31.6

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
2	01/14/80	S	265.0	C	4		RC	FUELXX	BEGAN CYCLE 4 ON 05/12/80 AT 0100
3	05/14/80	S	1.3	B	1		HA	TURBIN	TURBINE TRIP TESTING.
4	05/29/80	F	27.4	A	1		HA	MOTORX	LOW EHC OIL PRESSURE RESULTED IN MANUAL SHUTDOWN OF UNIT UNTIL EHC SYSTEM REPAIRED.
5	05/30/80	F	7.6	A	3		EB	XXXXXX	ELECTRICAL PROBLEM AFFECTING MAIN CIRC WATER PUMPS RESULTED IN LOSS OF CONDENSER VACUUM AND SUBSEQUENT TRIP.

SUMMARY

THE UNIT RETURNED TO SERVICE ON 05/12/80 AFTER REFUELING. THE UNIT RAN NEAR FULL POWER EXCEPT FOR MANUAL TRIPS ON 5/14, 5/29, AND 05/30/80 FOR TURBINE TESTING, LOW EHC OIL PRESSURE AND MAIN CIRC WATER PROBLEMS.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

ROBINSON 2

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	South Carolina/Darlington 5miles NW of Hartsville
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2300
5.	Design Electrical Rating (MWE Net)	700
6.	Date of Initial Criticality	September 20, 1970
7.	Date of Commercial Operation	March 7, 1971
8.	Condenser Cooling Method	Recirculation from impounded lake
9.	Condenser Cooling Water Source	Robinson Impoundment

Utility & Contractor Information

10.	Licensee	Carolina Power and Light Company
11.	Corporate Address	411 Fayetteville Street Raleigh, North Carolina
12.	Corporate Contact	J. A. Jones Executive Vice President
13.	Architect/Engineer	Ebasco
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Ebasco

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R.C. Lewis
18.	IE Section Chief	P. J. Kellogg
19.	IE Resident Inspector	R. Sauer
20.	Licensing Project Manager	D. Neighbors
21.	Docket Number	50-261
22.	License Number/Date of Issue	DPR-23, 7/31/70 (5 Mwt), Full power authorization issued 9/23/70
23.	Public Document Room	Hartsville Memorial Library Home and Fifth Avenue Hartsville, South Carolina 29550

INSPECTION STATUS

ENFORCEMENT STATUS

FAILURE TO PROVIDE ALARM CAPABILITY FOR VITAL AREA PORTAL.
(8009 INFRACTION)

FAILURE TO SECURE OPENINGS IN VITAL AREA BARRIER.
(8009 INFRACTION)

INOPERATIVE TAMPER ALARMS.
(8009 INFRACTION)

FAILURE TO PROVIDE CCTV SURVEILLANCE OF VITAL AREA.
(8009 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

UNIT IS IN NORMAL OPERATION.

LAST IE SITE INSPECTION DATE: MARCH 31 - APRIL 3, 1980

INSPECTION REPORT NO: 50-261/80-07

SUMMARY

INSPECTION SUMMARY

INSPECTION APRIL 21-25 (80-09): INCLUDED REVIEW OF THE LICENSEE'S SECURITY PROGRAM AUDIT, VITAL AND PROTECTED AREA BARRIERS, SECURITY LIGHTING, COMMUNICATIONS, DETECTION AIDS, ASSESSMENT AIDS, POWER SUPPLY, AND LOCKS, KEYS, AND COMBINATIONS. THE INSPECTION INVOLVED 60 INSPECTOR-HOURS ON SITE BY TWO NRC INSPECTORS. TEN INSPECTION HOURS WERE ACCOMPLISHED DURING OFFSHIFT PERIODS. THE LICENSEE WAS FOUND TO BE IN COMPLIANCE WITH NRC REQUIREMENTS WITHIN THE NINE AREAS EXAMINED DURING THE INSPECTION EXCEPT FOR THE FOLLOWING FOUR ITEMS: (1) PHYSICAL BARRIER - INSECURE GRATES AND HATCHES PROVIDED UNAUTHORIZED ACCESS INTO A VITAL AREA; (2) DETECTION AIDS - AN EXIT DOOR IN A VITAL AREA WAS NOT EQUIPPED WITH AN INTRUSION DETECTION ALARM; (3) DETECTION AIDS - TAMPER INDICATING CAPABILITY COULD NOT BE DEMONSTRATED IN A PROTECTED AREA ALARM SYSTEM; AND (4) ASSESSMENT AIDS - OPERABLE CCTV EQUIPMENT DEDICATED TO SURVEILLANCE OF A VITAL AREA DID NOT PROVIDE AN ACCEPTABLE LEVEL OF OBSERVATION.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

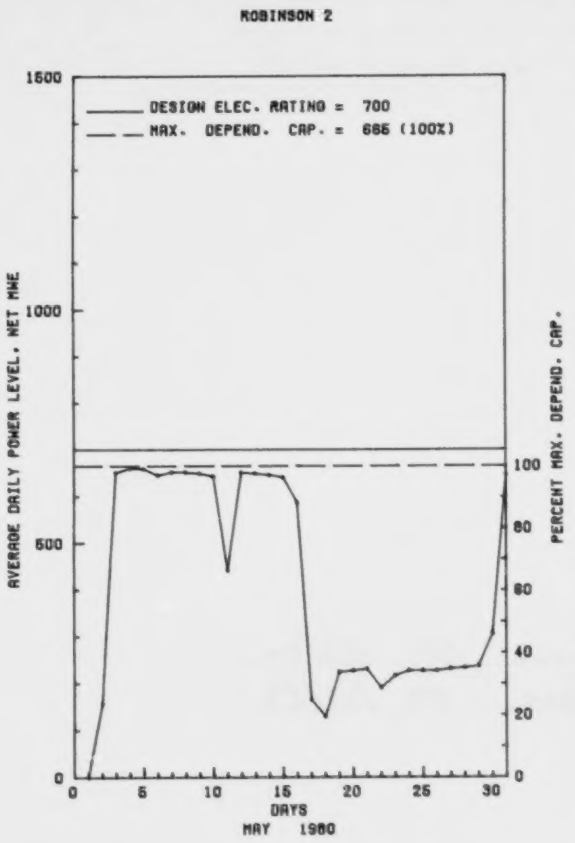
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-004/ 03L-0	03/16/80	04/14/80	LOSS OF MCC-5 AND MCC-10 DURING AN OUTAGE CAUSED BY INCORRECT OVERCURRENT SETTING OF MCC-5.
80-005/ 03L-0	03/20/80	04/19/80	REACTOR COOLANT PUMP THERMAL BARRIER OUTLET CONTAINMENT ISOLATION VALVE FAILED TO CLOSE.
80-007/ 03L-0	03/26/80	04/21/80	FIRST STAGE PRESSURE TRANSMITTERS ON TURBINE OUT OF CALIBRATION (ONE PERCENT DRIFT).
80-009/ 01T-0	04/14/80	04/15/80	PRIMARY TO SECONDARY LEAK "B" STEAM GENERATOR.

REVIEWED BY OMPA LOEB: L Tomson **DATE:** 7/2/80

REVIEWED BY NRR DOR: Don Neighbors **DATE:** 7/7/80

ROBINSON 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-261
2. Utility Contact: M. L. Watford (803) 383-4524
3. Licensed Thermal Power (MWt): 2300
4. Nameplate Rating (Gross MWe): 854 x 0.9 = 769
5. Design Electrical Rating (Net MWe): 700
6. Maximum Dependable Capacity (Gross MWe): 700
7. Maximum Dependable Capacity (Net MWe): 665
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): 2200 MWT
10. Reasons for Restrictions, If Any:
EXCESSIVE MOISTURE CARRY-OVER TO H.P. TURBINE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	3,647.0	81,029.0
12. Hours Reactor Was Critical	696.3	2,874.8	63,081.7
13. Reactor Reserve Shutdown Hours	43.1	200.6	919.9
14. Hours Generator On-Line	687.4	2,860.6	61,551.5
15. Unit Reserve Shutdown Hours	.0	.0	23.2
16. Gross Thermal Energy (MWH)	1,070,990	5,657,999	126,208,328
17. Gross Electrical Energy (MWH)	326,721	1,830,161	40,766,674
18. Net Electrical Energy (MWH)	304,600	1,729,117	38,636,219
19. Unit Service Factor	92.4	78.4	76.0
20. Unit Availability Factor	92.4	78.4	76.0
21. Unit Capacity Factor (MDC Net)	61.6	71.3	71.7
22. Unit Capacity Factor (DER Net)	58.5	67.7	68.1
23. Unit Forced Outage Rate	7.6	21.6	13.6

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
REFUELING/MAINTENANCE, JULY, SIX (6) WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
04-01	04/13/80	F	39.9	A	4		CB	PUMPXX	CONCLUSION OF OUTAGE REPORTED ON PREVIOUS REPORT.
05-01	05/10/80	F	.0	A	5		HH	VALVEX	"A" STEAM GENERATOR FW REGULATING VALVE EXPERIENCED PACKING FAILURE. NO PARTICULAR CAUSE WAS FOUND FOR THE PACKING FAILURE. PACKING WAS REPLACED. PARTIAL LOAD REDUCTION.
05-02	05/17/80	F	14.0	A	1		SH	INSTRU	WHILE REDUCING POWER TO CONSERVE FUEL AND PROLONG CORE LIFE, THE RADIAL TILT LIMIT OF 1.02 WAS EXCEEDED. WITH N-42 INOPERABLE DUE TO A FAILED DETECTOR, THE TRIP SETPOINTS COULD NOT BE RESET WITH THE REACTOR CRITICAL. THE UNIT WAS SHUTDOWN AND THE FAILED DETECTOR WAS REPLACED.
05-03	05/22/80	F	2.7	H	3		SH	INSTRU	REACTOR TRIP OCCURRED WHILE INSTRUMENTATION AND CONTROL TECHNICIANS SHORTED-OUT REACTOR COOLANT PUMP BEARING TEMPERATURE RECORDER.

SUMMARY

THE UNIT RETURNED TO OPERATION 05/02/80 AND RAN ROUTINELY EXCEPT FOR OUTAGES ON 05/17/80 AND ON 05/22/80 FOR DETECTOR REPLACEMENT AND AN INSTRUMENTATION SHORT. A POWER REDUCTION OCCURRED FROM 05/10/80 TO 05/30/80 FOR VALVE PACKING REPLACEMENT.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

SALEM 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	New Jersey/Salem 20 miles S of Wilmington, Del.
2.	Electric Reliability Council	Mid-Atlantic Area Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	3338
5.	Design Electrical Rating (MWE Net)	1090
6.	Date of Initial Criticality	December 11, 1976
7.	Date of Commercial Operation	June 30, 1977
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Delaware River

Utility & Contractor Information

10.	Licensee	Public Service Electric & Gas Co.
11.	Corporate Address	80 Park Place Newark, New Jersey 07101
12.	Corporate Contact	F. P. Librizzi General Manager - Electric Prod.
13.	Architect/Engineer	PSE&G
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	UE&C

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	IE Section Chief	R. R. Keimig
19.	IE Resident Inspector	L. J. Norrholm (609-935-3850) (FTS 341-3000)
20.	Licensing Project Manager	W. Ross
21.	Docket Number	50-272
22.	License Number/Date of Issue	DPR-70, 8/13/76
23.	Public Document Room	Salem Free Public Library 112 West Broadway Salem, New Jersey 08079

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

Spent fuel pool weld channel leakage of 200-300 ml/hr is being observed. Permanent repairs will be completed in June.

FACILITY ITEMS (PLANS AND PROCEDURES):

The next refueling outage is scheduled for September 20, 1980.

PLANT STATUS:

The unit is operating at full power.

LAST IE SITE INSPECTION DATE: 5/31/80

INSPECTION REPORT NO: 50-272/80-12

SUMMARY

INSPECTION SUMMARY

50-272/80-04 - Jan 28-31, Feb 1, 4-7 and 11-13: Routine, unannounced inspection by regionally-based inspectors and an NRR engineer of the Quality Assurance Program (QAP) implementation in the areas of QA/QC administration; design changes and modifications; document/drawing control; audits; QAP changes; and, followup on previously identified items. The inspection involved 124 inspector-hours onsite and 48 inspector-hours at the corporate offices by two NRC regional-based inspectors and one NRC headquarters (NRR-QAB) based engineer. Three items of noncompliance were identified: (Failure to develop/establish appropriate instructions/procedures; Failure to implement/follow established procedures; and Failure to store records in accordance with established requirements).

50-272/80-10 - Apr 22-25: Routine, unannounced inspection by one regionally-based inspector (32 hrs) of Containment Leak Rate Testing; inservice testing of pumps; surveillance of pipe supports and restraints; and licensee action on previous inspection findings. One item of noncompliance was identified: (Failure to meet CLRT reporting requirements).

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

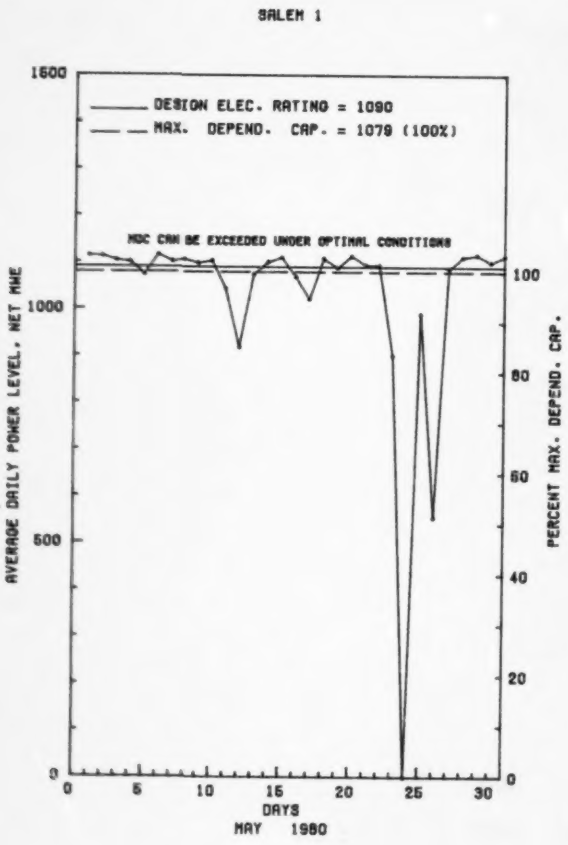
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-018/03L	04/07/80	05/05/80	No. 12 steam generator high steam flow went into continuous alarm. Cause was due to a failed signal isolator. Channel was placed in trip.
80-019/03L	04/09/80	05/12/80	Missed rod position surveillance with deviation monitor inoperable
80-020/03L	04/11/80	05/12/80	Auxiliary feedwater system manual isolation valve (14AF10) out of position
80-021/03L	04/17/80	05/16/80	Failure to verify operability of 2 service water loops during January & February 1980

REVIEWED BY OMPA LOEB: L. Tomason **DATE:** 7/3/80

REVIEWED BY NRR DOR: WJRoss **DATE:** 7/9/80

SALEM 1

AVERAGE DAILY POWER LEVEL(MWe)



OPERATING STATUS

1. Docket Number: 50-272
2. Utility Contact: L. K. Miller (609) 365-7000
3. Licensed Thermal Power (MWt): 3338
4. Nameplate Rating (Gross MWe): 1300 x 0.9 = 1170
5. Design Electrical Rating (Net MWe): 1090
6. Maximum Dependable Capacity (Gross MWe): 1124
7. Maximum Dependable Capacity (Net MWe): 1079
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	3,647.0	25,608.0
12. Hours Reactor Was Critical	720.0	3,532.5	13,620.8
13. Reactor Reserve Shutdown Hours	.0	.0	22.7
14. Hours Generator On-Line	712.4	3,425.7	12,956.8
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	2,320,798	10,566,170	38,102,466
17. Gross Electrical Energy (MWH)	795,480	3,583,020	12,790,520
18. Net Electrical Energy (MWH)	762,859	3,430,589	12,060,179
19. Unit Service Factor	95.8	93.9	50.6
20. Unit Availability Factor	95.8	93.9	50.6
21. Unit Capacity Factor (MDC Net)	95.0	87.2	43.6
22. Unit Capacity Factor (DER Net)	94.1	86.3	43.2
23. Unit Forced Outage Rate	4.2	6.1	38.0

24. Shutdowns Scheduled Over Next 6 Months (Type,Date,and Duration of Each):
REFUELING - 09/20/80 - 12/07/80.
25. If S'ut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-008	05/11/80	F	.0	A	5		HH	FILTER	CLEAN #11 CONDENSATE PUMP SUCTION STRAINERS.
80-110	05/12/80	F	.0	A	5		HH	FILTER	CLEAN #11 HEATER DRAIN PUMP SUCTION STRAINERS.
80-112	05/16/80	F	.0	A	5		SD	VALVEX	CONTAINMENT PENETRATION - VALVE 15533 MALFUNCTION.
80-114	05/16/80	F	.0	A	5		HH	FILTER	CHANGE #11 CONDENSATE PUMP SUCTION STRAINERS.
80-116	05/23/80	F	31.6	A	3		CH	VALVOP	#11 STEAM GENERATOR FEEDWATER CONTROL MALFUNCTION.
80-118	05/25/80	F	.0	A	5		HH	FILTER	#11 HEATER DRAIN PUMP FLANGE LEAK.

SUMMARY

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

SAN ONOFRE 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	California/San Diego 5 miles S of San Clemente
2.	Electric Reliability Council	Western Systems Coordinating Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	1347
5.	Design Electrical Rating (MWE Net)	436
6.	Date of Initial Criticality	June 14, 1967
7.	Date of Commercial Operation	January 1, 1968
8.	Condenser Cooling Method	Circulating sea water
9.	Condenser Cooling Water Source	Pacific Ocean

Utility & Contractor Information

10.	Licensee	Southern California Edison Co.
11.	Corporate Address	2244 Walnut Grove Avenue Rosemead, California 91770
12.	Corporate Contact	R. E. Woodbury V.P. & General Counsel
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	V
17.	IE Branch Chief	J. L. Crews
18.	IE Section Chief	B. H. Faulkenberry
19.	IE Resident Inspector	L. F. MILLER (714-492-2641)
20.	Licensing Project Manager	S. J. Nowicki
21.	Docket Number	50-206
22.	License Number/Date of Issue	DPR-13, 3/27/67
23.	Public Document Room	Mission Viejo Branch Library 24851 Chrisanta Drive Mission Viejo, California 92676

INSPECTION STATUS

ENFORCEMENT STATUS

LOGGING OF ENTRIES TO CERTAIN CONTROLLED AREAS.
(7712 DEFICIENCY)

CERTAIN PERSONS INADEQUATELY SEARCHED.
(7712 INFRACTION)

LESS THAN REQUIRED NUMBER OF GUARDS ONSITE.
(7712 INFRACTION)

ISOLATION ZONE NOT PROPERLY MAINTAINED.
(7712 DEFICIENCY)

IMPLEMENTATION SCHEDULE NOT SUBMITTED AS REQUIRED.
(7712 DEFICIENCY)

CERTAIN VEHICLES INADEQUATELY CONTROLLED WHILE ONSITE.
(7712 INFRACTION)

CERTAIN HANDCARRIED PACKAGES IMPROPERLY SEARCHED.
(7712 INFRACTION)

CERTAIN VEHICLES IMPROPERLY SEARCHED.
(7712 INFRACTION)

CERTAIN PERSONS IMPROPERLY BADGED.
(7712 INFRACTION)

10 CFR 50.55A, PARAGRAPH (G)(5), STATES, IN PART THAT "...COMPONENTS WHICH ARE CLASSIFIED AS ASME CODE CLASS 1, CLASS 2 AND CLASS 3 SHALL MEET THE REQUIREMENTS...SET FORTH IN SECTION XI...." THE APPLICABLE ADDENDA OF SECTION XI (SUMMER 1978) STATES THAT A REPAIR MAY BE MADE TO THE REQUIREMENTS OF SECTION XI OR TO THE REQUIREMENTS OF THE CODE OF CONSTRUCTION. THE CODE OF CONSTRUCTION FOR SAN ONOFRE UNIT 1 IS ANSI B31.1. SCE ELECTED TO USE ANSI B31.1-1977. CONTRARY TO THE REQUIREMENTS OF ANSI B31.1-1977, ON FEBRUARY 26, 1980 A MODIFIED SOCKET WELD FITTING WAS USED IN AN ASME CLASS 2 SYSTEM, THE CHARGING PUMP PIPING, LINE 2019-2-2502R, WITHOUT AN ANALYSIS OR THE SUBSTANTIATION AS REQUIRED BY PARAGRAPHS 106.1B AND 104.7.
(8004 INFRACTION)

APPENDIX B OF 10 CFR 50, CRITERION V, STATES, IN PART THAT "ACTIVITIES AFFECTING QUALITY SHALL BE PRESCRIBED BY DOCUMENTED INSTRUCTIONS, PROCEDURES AND SHALL BE ACCOMPLISHED IN ACCORDANCE WITH THESE INSTRUCTIONS, PROCEDURES...." BECHTEL WORK PLAN PROCEDURE NO. 19.20, "CONTROL OF WELDING FILLER MATERIAL," STATES, IN PART THAT IN CONTROLLED RADIATION AREAS, PARAGRAPH 6.1.12.1, "UNUSED MATERIAL SHALL BE DISPOSED OF BY BENDING...." CONTRARY TO THESE REQUIREMENTS, THE RESIDENT INSPECTOR FOUND SIX RODS OF E308L WELD FILLER MATERIAL ON THE FLOOR OF THE CHARGING ROOM ON FEBRUARY 27, 1980. NO WORK WAS IN PROGRESS IN THE CHARGING ROOM AT THE TIME. APPARENTLY, THE FILLER MATERIAL WAS LEFT FROM PRIOR WELDING ACTIVITIES AND HAD NOT BEEN DISPOSED OF IN ACCORDANCE WITH THE PROCEDURE.
(8004 INFRACTION)

TECHNICAL SPECIFICATION 6.9.2.B(1) REQUIRES THAT REACTOR PROTECTION SYSTEM INSTRUMENT SETTINGS WHICH ARE FOUND TO BE LESS CONSERVATIVE THAN THOSE ESTABLISHED BY THE TECHNICAL SPECIFICATIONS, BUT WHICH DO NOT PREVENT THE FULFILLMENT OF THE FUNCTIONAL REQUIREMENTS OF AFFECTED SYSTEMS, SHALL BE THE SUBJECT OF WRITTEN REPORTS TO THE DIRECTOR OF THE APPROPRIATE REGIONAL OFFICE WITHIN 30 DAYS OF OCCURRENCE OF THE EVENT. CONTRARY TO THESE REQUIREMENTS, A DIGITAL FEEDWATER FLOWMETER WAS INSTALLED AT THE FACILITY AND APPARENTLY USED BY SOME OPERATORS IN THE CALCULATION OF NUCLEAR POWER AT INTERVALS FROM OCTOBER 24, 1979 UNTIL DECEMBER 3, 1979. WHEN THIS USE WAS DISCOVERED, THE DIGITAL METER WAS INDICATING A VALUE OF FEEDWATER FLOW ONE TO TWO PERCENT LESS CONSERVATIVE THAN THE VALUES INDICATED BY THE PRECISION BARTON DP METERS WHICH OPERATING INSTRUCTION S-3-3.13, "REACTOR POWER CALCULATION," STATES "SHOULD BE USED." THIS DISCOVERY WAS NOT REPORTED IN ACCORDANCE WITH TECHNICAL SPECIFICATION 6.9.2.B(1).
(8004 DEFICIENCY)

SEE CONTINUATION PAGE S-5A

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT

NONE

TECHNICAL SPECIFICATION 3.3.1A.(1)H STATES THAT THE REACTOR SHALL NOT BE MAINTAINED CRITICAL UNLESS TWO SALT WATER COOLING PUMPS ARE OPERABLE OR ONE SALT WATER COOLING PUMP AND THE AUXILIARY SALT WATER COOLING PUMP ARE OPERABLE. TECHNICAL SPECIFICATION 3.3.1B(6) STATES THAT DURING CRITICAL OPERATION MAINTENANCE SHALL BE ALLOWED ON ONE OF THE TWO REQUIRED SALT WATER COOLING PUMPS OR AUXILIARY SALT WATER COOLING PUMP FOR A PERIOD OF TIME NOT LONGER THAN 72 CONSECUTIVE HOURS. CONTRARY TO THESE REQUIREMENTS, ON MARCH 10, 1980 FROM 9:25 P.M. TO 9:56 P.M. THE REACTOR WAS OPERATED AT OR NEAR 100 PERCENT OF RATED POWER WITH THE TWO SALT WATER COOLING PUMPS AND THE AUXILIARY SALT WATER COOLING PUMP INOPERABLE. ALSO, FROM 9:56 P.M. ON MARCH 10, 1980 TO 12:05 A.M. ON MARCH 11, 1980, THE REACTOR WAS OPERATED AT OR NEAR 100 PERCENT OF RATED POWER WITH BOTH SALT WATER COOLING PUMPS INOPERABLE.
(8009 INFRACTION)

TECHNICAL SPECIFICATION 6.8.1 STATES, IN PART THAT "WRITTEN PROCEDURES SHALL BE ESTABLISHED, IMPLEMENTED, AND MAINTAINED THAT MEET OR EXCEED THE REQUIREMENTS AND RECOMMENDATIONS OF SECTIONS 5.1 AND 5.3 OF ANSI N18.7-1976, 'ADMINISTRATIVE CONTROLS FOR NUCLEAR POWER PLANTS;' APPENDIX A OF USNRC REGULATORY GUIDE 1.33, REVISION 1, 'QUALITY ASSURANCE PROGRAM REQUIREMENTS (OPERATION)'...." EMERGENCY OPERATING INSTRUCTION S-3-5.34, REVISION 0, FEBRUARY 1, 1980, SECTION 3, STATES THAT IF BOTH SALT WATER COOLING PUMPS BECOME INOPERABLE AND THE AUXILIARY SALT WATER COOLING PUMP COOLING IS INADEQUATE, TRIP THE UNIT. CONTRARY TO THE ABOVE REQUIREMENTS, WHILE OPERATING THE REACTOR AT OR NEAR 100 PERCENT OF RATED POWER ON MARCH 10, 1980, THE UNIT WAS NOT TRIPPED WHEN BOTH SALT WATER COOLING PUMPS WERE INOPERABLE AND WHEN THE AUXILIARY SALT WATER COOLING PUMP WAS NOT PROVIDING ANY COOLANT FLOW TO THE SYSTEM.
(8009 INFRACTION)

TITLE 10, CODE OF FEDERAL REGULATIONS, PART 20.203.B, REQUIRES THAT EACH RADIATION AREA (DEFINED IN 10 CFR 20.202) SHALL BE CONSPICUOUSLY POSTED WITH A SIGN OR SIGNS BEARING THE RADIATION CAUTION SYMBOL AND THE WORDS: "CAUTION: RADIATION AREA." CONTRARY TO THIS REQUIREMENT, ON APRIL 17, 1980, RADIATION DOSE READINGS OF 8 MILLIREM PER HOUR WERE MEASURED OUTSIDE A POSTED TEMPORARY ROPE BARRIER SURROUNDING A SPENT RESIN CASK LOCATED ON A TRUCK. THE 8 MILLIREM PER HOUR RADIATION AREA WAS NOT CONSPICUOUSLY POSTED AS REQUIRED BY 10 CFR PART 20.203.B.
(8012 INFRACTION)

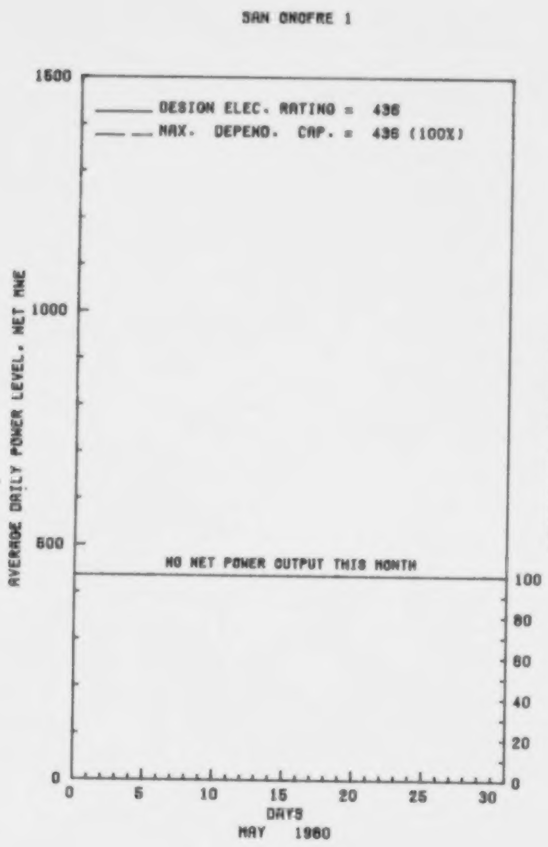
TECHNICAL SPECIFICATION 6.8.1 REQUIRES THAT WRITTEN PROCEDURES AND ADMINISTRATIVE POLICIES SHALL BE ESTABLISHED, IMPLEMENTED AND MAINTAINED THAT MEET OR EXCEED THE REQUIREMENTS AND RECOMMENDATIONS OF APPENDIX A OF USNRC REGULATORY GUIDE 1.33, REV. 1. PARAGRAPH 9.C.(7) OF REGULATORY GUIDE 1.33 RECOMMENDS WRITTEN PROCEDURES FOR THE REPLACEMENT OF NEUTRON DETECTORS. THE FACILITY TEST PROCEDURE NO. S-II.1.11, REF. 2, FOR REPLACING NEUTRON DETECTORS WAS WRITTEN PURSUANT TO THESE REQUIREMENTS. CONTRARY TO THE ABOVE REQUIREMENTS: (1) ON APRIL 16, 1980, FACILITY TEST PROCEDURE NO. S-II.1.11 WAS NOT IMPLEMENTED IN THE REPLACEMENT OF THE SOURCE RANGE INSTRUMENTATION DETECTORS AND (2) ON APRIL 24, 1980, THE DETECTORS FOR THE SOURCE RANGE INSTRUMENTATION BECAME ERRATIC AND WERE REPLACED PURSUANT TO THE REQUIRED PROCEDURE WITH THE EXCEPTION THAT HIGH VOLTAGE PLATEAU CURVES WERE NOT PREPARED AND USED AS REQUIRED BY STEP R OF THE PROCEDURE.
(8012 INFRACTION)

REVIEWED BY OMPA LOEB: Bobe **DATE:** 7/3/80

REVIEWED BY NRR DOR: for S.J. Nowicki **DATE:** 7/3/80

SAN ONOFRE 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-206
2. Utility Contact: R. G. Moran (714) 492-7700
3. Licensed Thermal Power (MWt): 1347
4. Nameplate Rating (Gross MWe): 500 x 0.9 = 450
5. Design Electrical Rating (Net MWe): 436
6. Maximum Dependable Capacity (Gross MWe): 456
7. Maximum Dependable Capacity (Net MWe): 436
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	3,647.0	113,623.0
12. Hours Reactor Was Critical	.0	1,979.1	84,611.3
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	.0	1,961.6	81,114.0
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	2,552,787	104,084,261
17. Gross Electrical Energy (MWH)	0	855,000	35,526,434
18. Net Electrical Energy (MWH)	0	816,676	33,664,042
19. Unit Service Factor	.0	53.8	74.8
20. Unit Availability Factor	.0	53.8	74.8
21. Unit Capacity Factor (MDC Net)	.0	51.4	72.0
22. Unit Capacity Factor (DER Net)	.0	51.4	72.0
23. Unit Forced Outage Rate	.0	2.8	9.2

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): ROUTINE REFUELING IN PROGRESS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: 07/01/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
55	04/09/80	S	744.0	C	4		RC	FUELXX	REFUELING.

SUMMARY

THE UNIT WAS SHUTDOWN FOR REFUELING.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

ENFORCEMENT STATUS

TECHNICAL SPECIFICATION 6.10.1.B REQUIRES THAT RECORDS AND LOGS OF PRINCIPAL MAINTENANCE ACTIVITIES, INSPECTIONS, REPAIR AND REPLACEMENT OF PRINCIPAL ITEMS OF EQUIPMENT RELATED TO NUCLEAR SAFETY BE RETAINED FOR A PERIOD OF AT LEAST FIVE YEARS. CONTRARY TO THIS REQUIREMENT, THE RECORDS REQUIRED BY TEST PROCEDURE S-II.1.11 RELATING TO REPLACEMENT OF DETECTORS FOR THE POWER RANGE INSTRUMENTATION, CHANNEL NO. 1205, ON FEBRUARY 4, 1980 WERE UNAVAILABLE ON APRIL 2 AND APPARENTLY NOT RETAINED AS REQUIRED.
(8012 DEFICIENCY)

TECHNICAL SPECIFICATION SECTION 6.8.1 REQUIRES THAT WRITTEN PROCEDURES "..BE ESTABLISHED, IMPLEMENTED AND MAINTAINED...." RADIATION PROTECTION PROCEDURE, S-VII-1.5, "ACCESS TO CONTROLLED AND EXCLUSION AREAS," STATES IN PART, "PLASTIC AND OTHER GLOVES MUST BE REMOVED AND HANDS MUST BE MONITORED BEFORE SMOKING OR DRINKING." CONTRARY TO THIS REQUIREMENT, ON APRIL 25 TWO INDIVIDUALS WERE OBSERVED TO BE SMOKING IN THE TURBINE DECK "DRINKING WATER-SMOKING AREA" WHILE WEARING GLOVES. ONE INDIVIDUAL WAS WEARING COTTON LINER GLOVES AND THE OTHER WAS WEARING BOTH COTTON LINER AND PLASTIC GLOVES.
(8013 INFRACTION)

INSPECTION STATUS

INSPECTION STATUS

SYSTEMS AND COMPONENT PROBLEMS:

Refueling outage commenced two days earlier than scheduled due to increase in primary to secondary (S/G tube) leakage (PNO-V-80-24).

FACILITY ITEMS (PLANS AND PROCEDURES):

Refueling.

MANAGERIAL ITEMS

None.

PLANT STATUS:

Plant shut down for refueling outage.

LAST IE SITE INSPECTION DATE: 05/01-30/80+

INSPECTION REPORT NO: 50-206/80-16+

SUMMARY

INSPECTION SUMMARY

Inspection on March 1-April 4, 1980 (Report No. 50-206/80-10)
Report being prepared; to be reported next month.

Inspection on April 21-25, 1980 (Report No. 50-206/80-11)
Areas Inspected: Routine, announced inspection of inservice inspection activities; maintenance and repair activities on pressurizer code safety valves; and licensee activities in response to IE Bulletin 79-02, "Pipe Support Base Plate Designs Using Concrete Expansion Anchor Bolts." The inspection included selective examinations of procedures and representative records; interviews with personnel; observation of work activities; and a review of licensee action on previous items of noncompliance. The inspection involved 64 inspector hours onsite by two NRC inspectors.

Results: No items of noncompliance or deviations were identified.

Inspection on April 5-May 2, 1980 (Report No. 50-206/80-12)
Areas Inspected: Routine resident inspection of plant operations as related to long term shutdown; monthly maintenance observations; long term shutdown activities; small break loss of coolant (SBLOCA) procedures; followup on licensee event reports; followup on licensee's response to IE Bulletin 79-15 and IE Circular 80-02. The inspection involved 67 inspector hours by one NRC inspector.

Results: In the monthly maintenance observation area, two items of noncompliance were identified concerning nuclear instrument detector replacements (Infraction - Failure to follow procedure; Deficiency - Failure to retain records). In the plant operations area, one item of noncompliance was identified (Infraction - Failure to implement fully the 10 CFR 20 posting requirements for a radiation area).

Inspection on April 23-25, 1980 (Report No. 50-206/80-13)
Areas Inspected: An alleger was interviewed at an offsite location, and nine specific allegations were examined at the licensee's facilities. The inspection involved 26 inspector hours onsite by two NRC inspectors.

Results: The inspector observed a failure to follow a radiation protection procedure which is required by Technical Specification 6.8.

Inspection on April 15-18, 1980 (Report No. 50-206/80-14)
Areas Inspected: A special inspection was conducted to determine the circumstances and events that occurred at the San Onofre Unit 1 nuclear facility when nine SCE employees were found to have radioactive contamination on their skin upon exiting the exclusion area and removing protective clothing. The inspection involved 44 hours onsite by two NRC Radiation Specialist inspectors.

Results: As a result of this inspection, one item of noncompliance with Technical Specification 6.11 requirements was identified.

Inspection on May 28-30, 1980 (Report No. 50-206/80-15)
Report being prepared; to be reported next month.

Inspection on May 1-30, 1980 (Report No. 50-206/80-16)
Report being prepared; to be reported next month.

Inspection on May 19-30, 1980 (Report No. 50-206/80-17)
Report being prepared; to be reported next month.

Inspection on May 19-22, 1980 (Report No. 50-206/80-18)
Report being prepared; to be reported next month.

ST. LUCIE 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Florida/St. Lucie 12 miles SE of Ft. Pierce
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2560
5.	Design Electrical Rating (MWE Net)	802
6.	Date of Initial Criticality	April 22, 1976
7.	Date of Commercial Operation	December 21, 1976
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Atlantic Ocean

Utility & Contractor Information

10.	Licensee	Florida Power & Light Co.
11.	Corporate Address	9250 W. Flagler Street P. O. Box 529100 Miami, Florida 33152
12.	Corporate Contact	R. E. Uhrig, V.P.
13.	Architect/Engineer	Ebasco
14.	Nuclear Steam Supply System	Combustion Engineering
15.	Constructor	Ebasco

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	H. C. LEWIS
18.	IE Section Chief	D. R. QUICK
19.	IE Resident Inspector	S. A. ELROD
20.	Licensing Project Manager	C. Nelson
21.	Docket Number	50-335
22.	License Number/Date of Issue	DPR-67, March 1, 1976
23.	Public Document Room	Indian River Junior College Library 3209 Virginia Avenue Ft. Pierce, Florida 33450

INSPECTION STATUS

ENFORCEMENT STATUS

AS REQUIRED BY 10 CFR 50, APPENDIX B, CRITERION V, ACTIVITIES AFFECTING QUALITY SHALL BE PRESCRIBED BY DOCUMENTED PROCEDURES AND SHALL BE ACCOMPLISHED IN ACCORDANCE WITH THESE PROCEDURES. THE LICENSEE'S ACCEPTED QA PROGRAM (FPLTQAR 1-76A), SECTION 5.1, REQUIRES THAT ACTIVITIES AFFECTING QUALITY SHALL BE ACCOMPLISHED IN ACCORDANCE WITH THESE PROCEDURES. SECTION 18.2.5 OF FPLTQAR 1-76A REQUIRES THE AUDITED ORGANIZATION TO PROVIDE A WRITTEN RESPONSE WITHIN 30 CALENDAR DAYS AFTER RECEIPT OF AN AUDIT REPORT. QP-18.1, SECTION 5.5.1 STATES THAT RESPONSES TO AUDIT FINDINGS ARE DUE WITHIN 30 CALENDAR DAYS AFTER RECEIPT OF THE AUDIT REPORT. QP-16-1, SECTIONS 5.4.3 AND 5.4.4 STATE THE ACTIONS TO BE TAKEN WHEN NO RESPONSE IS RECEIVED. THE RESPONSETO AUDIT FINDING #3 OF QAE-QAD-80-1 THAT WAS ISSUED ON FEBRUARY 20, 1980 HAD NOT BEEN PREPARED AS OF APRIL 15, 1980, AND NONE OF THE FOLLOWUP ACTIONS SPECIFIED IN QP-16-1 HAD BEEN INITIATED.
(8007 DEFICIENCY)

THE TECHNICAL SPECIFICATIONS, SECTION 6.8.1, STATES IN PART THAT WRITTEN PRECEDURES SHALL BE ESTABLISHED, IMPLEMENTED AND MAINTAINED. AP0010124, CONTROL AND USE OF JUMPERS AND DISCONNECTED LEADS IN SAFETY RELATED SYSTEMS, SECTION 5.4 REQUIRES IN PART THAT THE I&C SUPERVISOR AND THE ASSISTANT SUPERINTENDENT ELECTRICAL MAINTENANCE ARE RESPONSIBLE FOR REVIEWING, WITH THE OPERATIONS SUPERVISOR THE DISCONNECTED LEAD AND TEMPORARY JUMPER LOG...PRIOR T....EACH REFUELING. THE I&C SUPERVISOR AND THE ASSISTANT SUPERINTENDENT ELECTRICAL MAINTENANCE AND THE OPERATIONS SUPERINTENDENT DID NOT REVIEW THE DISCONNECTED LEAD AND TEMPORARY JUMPER LOG PRIOR TO THE REFUELING THAT COMMENCED MARCH 15, 1980.
(8007 INFRACTION)

AS REQUIRED BY CRITERION V OF APPENDIX B TO 10 CFR 50 AND AS IMPLEMENTED BY PARAGRAPH 17.2.5 OF THE FSAR, "ACTIVITIES AFFECTING QUALITY SHALL BE PRESCRIBED BY DOCUMENTED INSTRUCTIONS, PROCEDURES, DRAWINGS---AND SHALL BE ACCOMPLISHED IN ACCORDANCE WITH THESE INSTRUCTIONS, PROCEDURES, AND DRAWINGS". PARAGRAPH 5.4.3.1 OF FP&L PROCEDURE QI 9 PR/PSL-2, THE APPLICABLE PROCEDURE FOR THE CONTROL OF NDE, AND PARAGRAPH 8 OF ASNT-SNT-TC-1A REQUIRE A YEARLY EYE EXAMINATION FOR NDE PERSONNEL. ON APRIL 11, 1980, NDE RECORDS REVEALED THAT FOR THE SITE LEVEL III EXAMINER, AN EYE EXAMINATION HAD NOT BEEN ADMINISTERED WITHIN THE REQUIRED YEARLY TIME FRAME.
(8010 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

NORMAL OPERATION.

LAST IE SITE INSPECTION DATE: JUNE 3-6, 1980

INSPECTION REPORT NO: 50-335/80-16

SUMMARY

INSPECTION SUMMARY

INSPECTION APRIL 15-18 (80-07): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 48 INSPECTOR-HOURS IN OFFICE AND 57 INSPECTOR-HOURS ON SITE IN THE AREAS OF QA/QC ADMINISTRATION; QA PROGRAM REVIEW; HOUSEKEEPING AND CLEANLINESS PROGRAM; ORGANIZATION AND ADMINISTRATION; QA AUDITS; RECEIPT STORAGE AND HANDLING OF EQUIPMENT AND MATERIAL; OFFSITE SUPPORT STAFF; DESIGN, DESIGN CHANGES AND MODIFICATIONS; DESIGN CHANGES AND MODIFICATIONS PROGRAM; TEST AND EXPERIMENTS PROGRAM; MAINTENANCE: AND PERSONNEL QUALIFICATION PROGRAM. OF THE TWELVE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN TEN AREAS; TWO APPARENT ITEMS OF NONCOMPLIANCE WERE FOUND IN TWO AREAS (INFRACTION - FAILURE TO FOLLOW PROCEDURE; AND DEFICIENCY - FAILURE TO FOLLOW PROCEDURES).

INSPECTION APRIL 8-11 (80-10): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 25 INSPECTOR-HOURS ON SITE IN THE AREAS OF INSERVICE INSPECTION (ISI); IE BULLETIN 79-13. OF THE TWO AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN ONE AREA; ONE ITEM OF NONCOMPLIANCE WAS FOUND IN ONE AREA (DEFICIENCY - FAILURE OF NDE EXAMINER TO MEET EYE EXAMINATION REQUIREMENTS).

INSPECTION APRIL 29 - MAY 2 (80-12): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 21 INSPECTOR-HOURS ON SITE IN THE AREAS OF SURVEILLANCE ACTIVITY ON PIPE SUPPORT AND RESTRAINT SYSTEMS, INCLUDING EXAMINATION OF PROCEDURE TEST RESULTS AND INSTALLED RESTRAINTS, AND EXAMINATION OF RECORDS. OF THE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION APRIL 30 - MAY 2 (80-14): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 17 INSPECTOR-HOURS ON SITE IN THE AREAS OF CALIBRATION OF TECHNICAL SPECIFICATION REQUIRED PLANT INSTRUMENTS AND MEASURING AND TEST EQUIPMENT, AND FOLLOWUP OF RECENT LER'S. OF THE TWO AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN ONE AREA; ONE ITEM OF NONCOMPLIANCE WAS FOUND IN ONE AREA (INFRACTION - FAILURE TO REPORT EMERGENCY DIESEL GENERATOR LOAD SEQUENCE TIMERS THAT DID NOT MEET TECHNICAL SPECIFICATION LIMITS DURING ROUTINE SURVEILLANCE TESTING).

REPORTS RECEIVED FROM LICENSEE

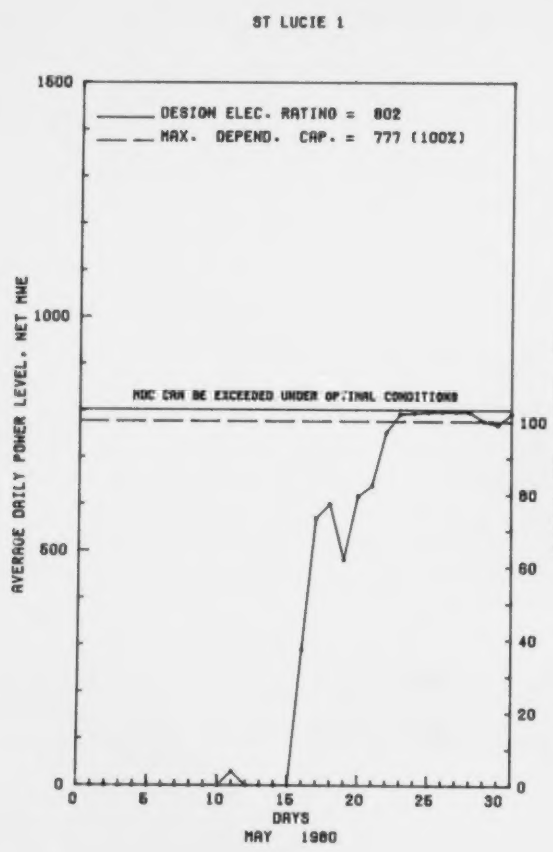
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-014/ 03L-0	03/24/80	04/23/80	HUMIDITY CONTROL HEATER FOR 6B2 SHIELD BUILDING VENT SYSTEM INOPERABLE DUE TO DEFECTIVE THERMOCOUPLE AMPLIFIER.
80-015/ 03L-0	03/16/80	04/15/80	CERTAIN ELECTRICAL ENGINEERED SAFETY FEATURES FAILED TO OPERATE DURING INTEGRATED SAFEGUARDS TEST.
80-016/ (PART 21)	04/11/80	04/16/80	ROSEMOUNT MODEL 1152 PRESSURE TRANSMITTER DESIGN PROBLEM.

REVIEWED BY OMPA LOEB: DATE: 7/3/80

REVIEWED BY NRR DOR: C Nelson DATE: 7/7/80

ST. LUCIE 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-335
2. Utility Contact: V. T. Chilson (305) 552-3824
3. Licensed Thermal Power (MWt): 2560
4. Nameplate Rating (Gross MWe): 1000 x 0.85 = 850
5. Design Electrical Rating (Net MWe): 802
6. Maximum Dependable Capacity (Gross MWe): 822
7. Maximum Dependable Capacity (Net MWe): 777
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	3,647.0	30,191.0
12. Hours Reactor Was Critical	497.1	2,280.2	23,790.1
13. Reactor Reserve Shutdown Hours	.0	.0	129.4
14. Hours Generator On-Line	411.6	2,192.2	23,022.3
15. Unit Reserve Shutdown Hours	.0	.0	32.0
16. Gross Thermal Energy (MWH)	890,501	5,377,784	55,637,350
17. Gross Electrical Energy (MWH)	284,880	1,734,530	18,029,030
18. Net Electrical Energy (MWH)	262,005	1,628,138	16,956,034
19. Unit Service Factor	55.3	60.1	76.3
20. Unit Availability Factor	55.3	60.1	76.4
21. Unit Capacity Factor (MDC Net)	45.3	57.5	72.3
22. Unit Capacity Factor (DER Net)	43.9	55.7	70.0
23. Unit Forced Outage Rate	.0	.7	5.0

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
4	03/15/80	S	232.2	C	4		RC	FUELXX	UNIT WAS REMOVED FROM SERVICE FOR SCHEDULED REFUELING, MAINTENANCE, AND INSPECTIONS.
5	05/11/80	S	3.7	B	1		HA	MECFUN	UNIT WAS REMOVED FROM SERVICE TO PERFORM SURVEILLANCE TEST ON TURBINE OVERSPEED TRIP DEVICE.
6	05/11/80	S	96.5	B	1		HA	MECFUN	UNIT WAS REMOVED FROM SERVICE TO TEST TURBINE OVERSPEED TRIP DEVICE & REPAIR TURBINE SHAFT SEAL #3. OUTAGE WAS EXTENDED TO REPLACE VALVE STEMS ON THE BYPASS VALVE AROUND MAIN STEAM ISOLATION VALVE NOS. 1A & 1B & REPAIR VALVE STEM PACKING LEAKS ON VALVES INSIDE CONTAINMENT THAT COULD NOT BE ISOLATED.
7	05/18/80	F	.0	B	5		HH	FILTER	LOAD REDUCTION TO APPROXIMATELY 50% R.P. TO CLEAN CONDENSATE PUMP SUCTION STRAINERS.

SUMMARY

THE PLANT OPERATED AT APPROXIMATELY 100% FOLLOWING A REFUELING AND TESTING OUTAGE EXCEPT FOR AN OUTAGE ON 05/11/80 FOR TURBINE SPIN TEST AND VALVE REPAIR AND A REDUCTION ON 05/18/80 FOR CONDENSATE PUMP CLEANING.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

SURRY 1

FACILITY DATA

Facility Description

1. Location (state/county) (nearest population center)	Virginia/Surry 17 miles NW of Newport News
2. Electric Reliability Council	Southeastern Electric Reliability Council
3. Type of Reactor	PWR
4. Power Level (MWT)	2441
5. Design Electrical Rating (MWE Net)	822
6. Date of Initial Criticality	July 1, 1972
7. Date of Commercial Operation	December 22, 1972
8. Condenser Cooling Method	Once Through
9. Condenser Cooling Water Source	James River

Utility & Contractor Information

10. Licensee	Virginia Electric and Power Company
11. Corporate Address	P. O. Box 26666 Richmond, Virginia 23261
12. Corporate Contact	J. H. Ferguson Exec. V.P. - Power
13. Architect/Engineer	Stone & Webster
14. Nuclear Steam Supply System	Westinghouse
15. Constructor	Stone & Webster

Regulatory Information

16. IE Region responsible	II
17. IE Branch Chief	R.C. Lewis
18. IE Section Chief	P. J. Kellogg
19. IE Resident Inspector	D. Burke (804) 357-2101
20. Licensing Project Manager	D. Neighbors
21. Docket Number	50-280
22. License Number/Date of Issue	DPR-32, 5/25/72
23. Public Document Room	Swem Library College of William and Mary Williamsburg, Virginia 23185

INSPECTION STATUS

ENFORCEMENT STATUS

WELDING OPERATIONS WERE BEING CONDUCTED ON THE 2 FOOT ELEVATION OF THE AUXILIARY BUILDING WITHOUT A WELDING AND FLAME PERMIT AS REQUIRED BY TECHNICAL SPECIFICATION 6.4.J AND VEPCO PROCEDURE ADM-56 SPECIAL PROCESSES INVOLVING IGNITION SOURCES.
(8008 INFRACTION)

VEPCO LETTER SERIAL NUMBER 725 DATED SEPTEMBER 5, 1979 STATED THAT "ACCESSIBLE SUPPORTS WILL BE UPGRADED EXPEDITIOUSLY AS DEFICIENCIES ARE NOTED DURING PLANT OPERATION...AS REQUIRED BY IE BULLETIN 79-02". THIS LETTER FURTHER STATED TH'T, "WE PLAN TOFULLY COMPLY WITH THE INTENT AND SCHEDULE OF BULLETIN 79-02...". A LETTER FROM REGION II TO VEPCO DATED SEPTEMBER 7, 1979 CONFIRMING ACTIONS TO BE TAKEN BY VEPCO RELATED TO RETURNING UNIT 1 TO SERVICE STATED THAT, "IT IS OUR UNDERSTANDING THAT AFTER YOU COMMENCE OPERATION OF UNIT 1, YOU WILL CONTINUE THE INSPECTIONS REQUIRED BY IEB 79-02 FOR ACCESSIBLE SYSTEMS". NRC INSPECTION REPORT 50-280/79-5E3 DATED SEPTEMBER 28, 1979 REITERATED THAT THE EXP EDITIOUS "COMPLETION OF THE REMAINING INSPECTIONS, TESTS, ANALYSES AND REPAIRS AS SPECIFIED IN IEB 79-02" WAS EXPECTED. AS OF APRIL 18, 1980 NONE OF THE ADDITIONAL INSPECTIONS OR TESTING REQUIRED FOR IE BULLETIN 79-02 HAD BEEN INITIATED. NO PROCEDURE HAD BEEN PREPARED FOR PERFORMANCE OF THIS WORK.
(8012 DEVIATION)

AS REQUIRED BY 10 CFR 50.59, THE LICENSEE MAY MAKE CHANGES IN THE FACILITY AS DESCRIBED IN THE SAFETY ANALYSIS REPORT (SAR) WITHOUT NRC APPROVAL, PROVIDED RECORDS OF THE CHANGES ARE MAINTAINED, INCLUDING WRITTEN SAFETY EVALUATIONS WHICH DETERMINE THAT THE CHANGE DOES NOT PRESENT AN UNREVIEWED SAFETY QUESTION. SAFETY EVALUATIONS AND RECORDS OR CERTAIN CHANGES MADE TO COMPONENTS DESCRIBED IN SECTION 6.3.1.4 OF THE SAR TO DETECT OR CONTROL LEAKS IN THE LHSI AND OUTSIDE RECIRCULATION SPRAY SUCTION PIPING WERE NOT MAINTAINED OR AVAILABLE ON MARCH 28, 1980. THE CHANGES INCLUDED THE VALVE PIT BAFFLES, WHICH WERE NOT INSTALLED TO ISOLATE THE SPRAY SUBSYSTEM LEAKAGE, AND THE PIT LIQUID LEVEL DETECTORS; ONLY ONE OF THE TWO LEVEL DETECTORS (PER UNIT) WERE INSTALLED.
(8013 INFRACTION)

AS REQUIRED BY 10 CFR 73.55(D)(7), THE LICENSEE SHALL POSITIVELY CONTROL ALL POINTS OF PERSONNEL AND VEHICLE ACCESS INTO VITAL AREAS. SECTION 5.3.1.3 OF THE SURRY POWER STATION SECURITY PLAN, WHICH IMPLEMENTS THE ABOVE, STATES, IN PART, THAT ALL VITAL AREA PORTALS ARE CONTROLLED BY ELECTRONIC LOCKS OR KEY LOCKS, OR BY ADMINISTRATIVE CONTROL OF SECURITY OR AUTHORIZED STATION OPERATING PERSONNEL. UNDER "DEFINITIONS" OF THE SECURITY PLAN, ADMINISTRATIVE CONTROL IS DEFINED AS THE MEANS BY WHICH AN ENTRANCE, GATE,...ETC., IS EITHER LOCKED AND ALARMED OR UNDER THE DIRECT SUPERVISION AND CONTROL OF AN INDIVIDUAL AUTHORIZED UNESCORTED ACCESS TO THE AREA.... ON NOVEMBER 28, 1979, THE REACTOR CONTROL ROOM DOOR WAS FOUND STANDING OPEN, THE ALARM SYSTEM WAS IN "ACCESS" (OUT OF SERVICE), AND THE ENTRANCE ACCESS WAS NOT UNDER THE DIRECT SUPERVISION AND CONTROL OF AN INDIVIDUAL AUTHORIZED UNESCORTED ACCESS TO THE AREA; THUS, ACCESS INTO THIS VITAL AREA WAS NOT POSITIVELY CONTROLLED. AUTHORIZED PERSONNEL WERE CONTROLLING ACCESS FROM INSIDE THE CONTROL ROOM WHILE THE ALARM AND ACCESS SYSTEM WAS INOPERABLE.
(8013 DEFICIENCY)

AS REQUIRED BY TECHNICAL SPECIFICATION 6.4.A.4, DETAILED WRITTEN PROCEDURES WITH APPROPRIATE CHECK-OFF LISTS AND INSTRUCTIONS SHALL BE PROVIDED FOR ACTIONS TO BE TAKEN FOR SPECIFIC AND FORESEEN MALFUNCTIONS OF SYSTEMS OR COMPONENTS INCLUDING ALARMS, PRIMARY SYSTEM LEAKS AND ABNORMAL REACTIVITY CHANGES. CONTRARY TO THE ABOVE, DETAILED WRITTEN PROCEDURES WITH APPROPRIATE CHECK-OFF LISTS AND INSTRUCTIONS HAVE NOT BEEN PROVIDED AS OF APRIL 25, 1980, FOR ACTIONS TO BE TAKEN WHEN CERTAIN INSTALLED RADIATION MONITORING COMPONENTS MALFUNCTION. CONSEQUENTLY, WHEN THE UNIT 2 RADIATION MONITOR (RM-RMS-262) ON THE MANIPULATOR CRANE INTERMITTENTLY FAILED ON APRIL 21 AND 22, 1980, DURING UNIT 2 REFUELING, APPROPRIATE PROMPT CORRECTIVE ACTION WAS NOT TAKEN TO CORRECT THE MALFUNCTION WHICH WAS INDICATED BY LOSS OF THE GREEN FAIL/RESET LIGHT AND MR/HR METER INDICATION. SIMILARLY, PROCEDURES DO NOT EXIST FOR ACTIONS TO BE TAKEN WHEN FLOW OR FILTER FAULT ALARMS OCCUR ON RM-GW-101 AND RM-159, 160, 259 AND 260.
(8019 INFRACTION)

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT

NONE

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

UNIT IS CURRENTLY OPERATING ROUTINELY.

LAST IE SITE INSPECTION DATE: JUNE 6, 1980

INSPECTION REPORT NO: 50-280/80-20

SUMMARY

INSPECTION SUMMARY

INSPECTION APRIL 15-17 (80-11): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 14 INSPECTOR-HOURS ON SITE IN THE AREAS OF STEAM GENERATOR SHIPMENT ACTIVITIES AND PREVIOUSLY IDENTIFIED ITEMS. OF THE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION APRIL 16-18 (80-12): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 12 INSPECTOR-HOURS ON SITE IN THE AREAS OF IE BULLETIN 79-14, "SEISMIC ANALYSIS FOR AS-BUILT SAFETY-RELATED PIPING SYSTEMS," AND IE BULLETIN 79-02, "PIPE SUPPORT BASE PLATE DESIGNS USING CONCRETE EXPANSION ANCHOR BOLTS." OF THE TWO AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN ONE AREA; ONE DEVIATION WAS FOUND IN ONE AREA (FAILURE TO MEET COMMITMENT FOR TIMELY PERFORMANCE OF IEB 79-02 INSPECTIONS).

INSPECTION APRIL 16-18 (80-14): THIS SPECIAL, ANNOUNCED INSPECTION INVOLVED 11 INSPECTOR-HOURS ON SITE IN THE AREA OF INSTALLED EQUIPMENT REVIEW WITH RESPECT TO THE IE BULLETIN 79-01B RESPONSE. OF THE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION APRIL 23 AND 25 (80-16): THIS SPECIAL, ANNOUNCED INSPECTION INVOLVED 10 INSPECTOR-HOURS ON SITE CONCERNING THE APRIL 14, 1980, SHIPMENT OF RADIOACTIVE WASTE FOR BURIAL INCLUDING SURVEYS, SHIPPING PAPER REQUIREMENTS, LABELING, PLACARDING AND RADIATION LEVELS. DETAILS I ADDRESSES INSPECTION ACTIVITIES AT THE WASTE DISPOSAL SITE. DETAILS II ADDRESSES INSPECTION ACTIVITIES AT SURRY POWER STATION. OF THE FIVE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN THREE AREAS; TWO ITEMS OF NONCOMPLIANCE WERE FOUND IN TWO AREAS (SEVERITY LEVEL II - VEHICLE SURFACE RADIATION LEVELS IN EXCESS OF REGULATORY LIMITS; AND SEVERITY LEVEL II - INADEQUATE SURVEY).

INSPECTION APRIL 29 - MAY 2 (80-18): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 14 INSPECTOR-HOURS ON SITE IN THE AREAS OF IE BULLETIN 79-02, "PIPE SUPPORT BASE PLATE DESIGN USING CONCRETE EXPANSION ANCHOR BOLTS"; IE BULLETIN 79-14, "SEISMIC ANALYSIS FOR AS-BUILT SAFETY-RELATED PIPING SYSTEMS"; AND LICENSEE ACTIONS ON PREVIOUS INSPECTION FINDINGS. OF THE THREE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED

INSPECTION APRIL 7 - MAY 2 (80-19): THIS ROUTINE INSPECTION BY THE RESIDENT INSPECTOR INVOLVED 50 INSPECTOR-HOURS ON SITE IN THE AREAS OF PLANT OPERATIONS AND OPERATING RECORDS, PLANT MODIFICATIONS, MAINTENANCE AND TESTING, FOLLOWUP ON PREVIOUSLY IDENTIFIED ITEMS, AND PLANT SECURITY. OF THE FOUR AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE FOUND IN TWO AREAS; TWO APPARENT ITEMS OF NONCOMPLIANCE WERE FOUND IN TWO AREAS (INFRACTION - INADEQUATE PROCEDURES FOR RADIATION MONITORING EQUIPMENT MALFUNCTIONS; AND INFRACTION - FAILURE TO FOLLOW TEST PROCEDURE FOR DC 77-9).

INSPECTION MAY 5-8 (80-20): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 13 INSPECTOR-HOURS ON SITE IN THE AREAS OF SCOPING THE SURRY 2 INTEGRATED STARTUP TEST PROGRAM AS TO THE INSPECTIONS AND TESTS REQUIRED PRIOR TO RETURNING THE UNIT TO OPERATION AFTER THE STEAM GENERATOR REPAIR PROGRAM, REVIEW THE COMPLETED RESULTS OF THE PRIMARY HYDROSTATIC TESTS AND REVIEW OF THE GENERAL ELECTRICAL, INSTRUMENT AND MECHANICAL TEST PROCEDURES. OF THE THREE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN TWO AREAS; ONE ITEM OF NONCOMPLIANCE WAS FOUND IN ONE AREA (INFRACTION - INADEQUATE TEST PROCEDURE).

REVIEWED BY OMPA LOEB:

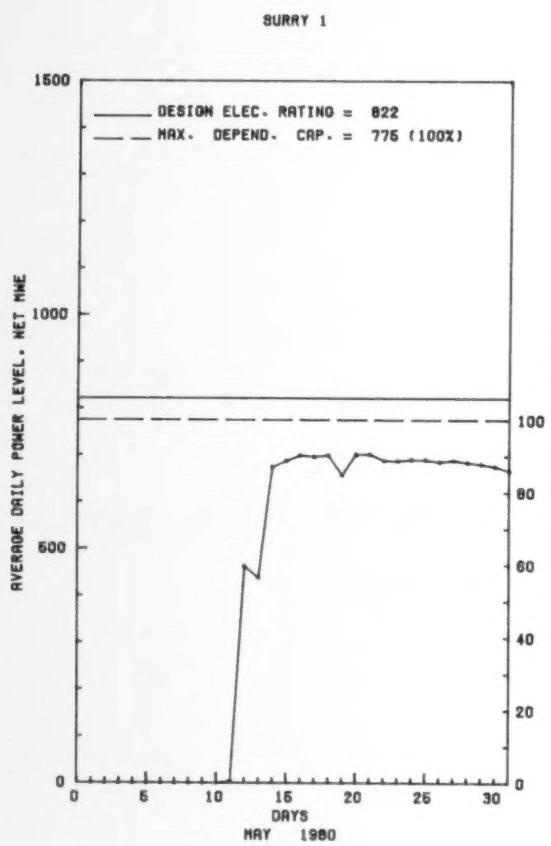
DATE: 7/3/80

REVIEWED BY NRR DOR: Don Neighbors

DATE: 7/7/80

SURRY 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-280
2. Utility Contact: O. J. Costello (804) 357-3184
3. Licensed Thermal Power (MWt): 2441
4. Nameplate Rating (Gross MWe): 942 x 0.9 = 848
5. Design Electrical Rating (Net MWe): 822
6. Maximum Dependable Capacity (Gross MWe): 811
7. Maximum Dependable Capacity (Net MWe): 775
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	3,647.0	65,231.0
12. Hours Reactor Was Critical	487.8	1,504.0	40,255.6
13. Reactor Reserve Shutdown Hours	.0	182.6	3,731.5
14. Hours Generator On-Line	481.2	1,491.6	39,396.6
15. Unit Reserve Shutdown Hours	.0	183.6	3,736.2
16. Gross Thermal Energy (MWH)	1,130,680	3,569,020	91,283,892
17. Gross Electrical Energy (MWH)	337,905	1,123,560	29,804,058
18. Net Electrical Energy (MWH)	319,190	1,066,000	28,292,899
19. Unit Service Factor	64.7	40.9	60.4
20. Unit Availability Factor	64.7	45.9	66.1
21. Unit Capacity Factor (MDC Net)	55.4	37.7	56.0
22. Unit Capacity Factor (DER Net)	52.2	35.6	52.8
23. Unit Forced Outage Rate	35.3	59.1	27.1

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): S/G REPLACEMENT AND REFUELING, 09/15/80; (1 Year)
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-7	02/19/80	F	260.9	D	4	80-018			CONTINUATION OF SHUTDOWN FOR PIPING RESTRAINT MODIFICATION AND TURBINE ROTOR INSPECTION WHICH BEGAN 02/19/80.
80-8	05/11/80	F	1.9	G	3				REACTOR TRIP CAUSED BY FEED FLOW/STEAM FLOW MISMATCH WITH LO S/G'S IN MANUAL DURING ESCALATION OF POWER.
80-9	05/12/80	F	.0	G	5				INADVERTANT BORATION OF PRIMARY SYSTEM WHEN "B" MIXED BED DEMINERALIZER WAS PLACED IN SERVICE WITHOUT VERIFYING BORON CONCENTRATION OF EFFLUENT WAS EQUALIZED WITH PRIMARY SYSTEM BORON CONCENTRATION. INADVERTANT BORATION CAUSED DELTA 0 TO REMAIN OUTSIDE TARGET BAND FOR GREATER THAN ONE HOUR IN A 24 HOUR PERIOD NECESSITATING POWER REDUCTION TO <50% POWER WITHIN TARGET BAND FOR 24 HOURS.

SUMMARY

THE UNIT RETURNED TO SERVICE FROM A PIPING MODIFICATION AND TURBINE INSPECTION OUTAGE ON 05/11/80. THE UNIT SUFFERED A TRIP DURING STARTUP AND THEN OPERATED ROUTINELY WITH ONE REDUCTION THE REST OF THE MONTH.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

SURRY 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Virginia/Surry 17 miles NW of Newport News
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2441
5.	Design Electrical Rating (MWE Net)	822
6.	Date of Initial Criticality	March 7, 1973
7.	Date of Commercial Operation	May 1, 1973
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	James River

Utility & Contractor Information

10.	Licensee	Virginia Electric and Power Company
11.	Corporate Address	P. O. Box 26666 Richmond, Virginia 23261
12.	Corporate Contact	J. H. Ferguson Exec. V.P. - Power
13.	Architect/Engineer	Stone & Webster
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R.C. Lewis
18.	IE Section Chief	P. J. Kellog
19.	IE Resident Inspector	D. Burke (804) 357-2101
20.	Licensing Project Manager	D. Neighbors
21.	Docket Number	50-281
22.	License Number/Date of Issue	DPR-37, 1/29/73
23.	Public Document Room	Swem Library College of William and Mary Williamsburg, Virginia 23185

INSPECTION STATUS

ENFORCEMENT STATUS

FIRE DOOR IN PENETRATION BETWEEN UNIT 2 EMERGENCY SWITCHGEAR ROOM AND SERVICE BUILDING CABLE VAULT WAS BLOCKED IN THE OPEN POSITION ON APRIL 17, 1980 BY ELECTRICAL CABLE. TECHNICAL SPECIFICATION SECTION 3.21.G REQUIRES FIRE BARRIER PENETRATIONS TO BE FUNCTIONAL AT ALL TIMES OR IF NON-FUNCTIONAL A FIRE WATCH MUST BE ESTABLISHED ON AT LEAST ONE SIDE OF THE AFFECTED PENETRATION.
(8009 INFRACTION)

VEPCO LETTER SERIAL NUMBER 725 DATED SEPTEMBER 5, 1979 STATED THAT "ACCESSIBLE SUPPORTS WILL BE UPGRADED EXPEDITIOUSLY AS DEFICIENCIES ARE NOTED DURING PLANT OPERATION...AS REQUIRED BY IE BULLETIN 79-02". THIS LETTER FURTHER STATED THAT, "WE PLAN TOFULLY COMPLY WITH THE INTENT AND SCHEDULE OF BULLETIN 79-02...". A LETTER FROM REGION II TO VEPCO DATED SEPTEMBER 7, 1979 CONFIRMING ACTIONS TO BE TAKEN BY VEPCO RELATED TO RETURNING UNIT 1 TO SERVICE STATED THAT, "IT IS OUR UNDERSTANDING THAT AFTER YOU COMMENCE OPERATION OF UNIT 1, YOU WILL CONTINUE THE INSPECTIONS REQUIRED BY IEB 79-02 FOR ACCESSIBLE SYSTEMS". NRC INSPECTION REPORT 50-280/79-5E3 DATED SEPTEMBER 28, 1979 REITERATED THAT THE EXP EDITIOUS "COMPLETION OF THE REMAINING INSPECTIONS, TESTS, ANALYSES AND REPAIRS AS SPECIFIED IN IEB 79-02" WAS EXPECTED. AS OF APRIL 18, 1980 NONE OF THE ADDITIONAL INSPECTIONS OR TESTING REQUIRED FOR IE BULLETIN 79-02 HAD BEEN INITIATED. NO PROCEDURE HAD BEEN PREPARED FOR PERFORMANCE OF THIS WORK.
(8013 DEVIATION)

AS REQUIRED BY 10 CFR 50.59, THE LICENSEE MAY MAKE CHANGES IN THE FACILITY AS DESCRIBED IN THE SAFETY ANALYSIS REPORT (SAR) WITHOUT NRC APPROVAL, PROVIDED RECORDS OF THE CHANGES ARE MAINTAINED, INCLUDING WRITTEN SAFETY EVALUATIONS WHICH DETERMINE THAT THE CHANGE DOES NOT PRESENT AN UNREVIEWED SAFETY QUESTION. SAFETY EVALUATIONS AND RECORDS OR CERTAIN CHANGES MADE TO COMPONENTS DESCRIBED IN SECTION 6.3.1.4 OF THE SAR TO DETECT OR CONTROL LEAKS IN THE LHSI AND OUTSIDE RECIRCULATION SPRAY SUCTION PIPING WERE NOT MAINTAINED OR AVAILABLE ON MARCH 28, 1980. THE CHANGES INCLUDED THE VALVE PIT BAFFLES, WHICH WERE NOT INSTALLED TO ISOLATE THE SPRAY SUBSYSTEM LEAKAGE, AND THE PIT LIQUID LEVEL DETECTORS; ONLY ONE OF THE TWO LEVEL DETECTORS (PER UNIT) WERE INSTALLED.
(8014 INFRACTION)

AS REQUIRED BY 10 CFR 73.55(D)(7), THE LICENSEE SHALL POSITIVELY CONTROL ALL POINTS OF PERSONNEL AND VEHICLE ACCESS INTO VITAL AREAS. SECTION 5.3.1.3 OF THE SURRY POWER STATION SECURITY PLAN, WHICH IMPLEMENTS THE ABOVE, STATES, IN PART, THAT ALL VITAL AREA PORTALS ARE CONTROLLED BY ELECTRONIC LOCKS OR KEY LOCKS, OR BY ADMINISTRATIVE CONTROL OF SECURITY OR AUTHORIZED STATION OPERATING PERSONNEL. UNDER "DEFINITIONS" OF THE SECURITY PLAN, ADMINISTRATIVE CONTROL IS DEFINED AS THE MEANS BY WHICH AN ENTRANCE, GATE....ETC, IS EITHER LOCKED AND ALARMED OR UNDER THE DIRECT SUPERVISION AND CONTROL OF AN INDIVIDUAL AUTHORIZED UNESCORTED ACCESS TO THE AREA.... ON NOVEMBER 28, 1979, THE REACTOR CONTROL ROOM DOOR WAS FOUND STANDING OPEN, THE ALARM SYSTEM WAS IN "ACCESS" (OUT OF SERVICE), AND THE ENTRANCE ACCESS WAS NOT UNDER THE DIRECT SUPERVISION AND CONTROL OF AN INDIVIDUAL AUTHORIZED UNESCORTED ACCESS TO THE AREA; THUS, ACCESS INTO THIS VITAL AREA WAS NOT POSITIVELY CONTROLLED. AUTHORIZED PERSONNEL WERE CONTROLLING ACCESS FROM INSIDE THE CONTROL ROOM WHILE THE ALARM AND ACCESS SYSTEM WAS INOPERABLE.
(8014 DEFICIENCY)

AS REQUIRED BY TECHNICAL SPECIFICATION 6.4.A.2 AND 6.4.D THE DETAILED WRITTEN PROCEDURES WITH APPROPRIATE CHECK-OFF LISTS FOR THE CALIBRATION AND TESTING OF INSTRUMENTS, COMPONENTS, AND SYSTEMS INVOLVING NUCLEAR SAFETY SHALL BE FOLLOWED. CONTRARY TO THE ABOVE, THE UNIT 2 CONTAINMENT SPRAY CHEMICAL ADDITION FLOW AND LHSI VENTURI FLOW VERIFICATION TEST, CONDUCTED ON APRIL 15, 1980, TO VERIFY DESIGN CHANGE 77-9 MODIFICATIONS, WAS NOT PROPERLY FOLLOWED OR COMPLETED. ON APRIL 15 AND 16, 1980, THE FOLLOWING DISCREPANCIES WERE NOTED: (1) THE INITIAL CONDITIONS OF THE TEST REQUIRED THAT THE SAFETY INJECTION AND CONTAINMENT SPRAY SYSTEM WERE AVAILABLE FOR SERVICE PER OP-7.1 AND OP-7.2 RESPECTIVELY. VALVE ALIGNMENT CHECKLISTS OP-7.1 AND 7.2 WERE NOT COMPLETED PRIOR TO THE TEST, BUT WERE "VISUALLY CHECKED". DURING THE TEST, THE LHSI PUMP 2-SI-P-1B WAS OPERATED FOR SOME 30 MINUTES WITH THE PUMP SUCTION VALVE INADVERTENTLY CLOSED; (2) STEP 6.1.1 OF THE TEST PROCEDURE REQUIRED THE RWST WATER PH TO BE BETWEEN 4.0 AND 4.5; THE ACTUAL VALVE WAS 5.4 AND CORRECTIVE ACTION, DEVIATIONS, OR CHANGES TO THE PROCEDURE WERE NOT MADE; (3) STEP 6.2.2 OF THE PROCEDURE IS THE FIRST OF SEVERAL STEPS FOR CAT FLOW VERIFICATION WHICH WERE CONDUCTED BUT NOT SIGNED AND DATED; (4) ATTACHMENT IV TO THE PROCEDURE, WHICH VERIFIED TEST INSTRUMENTATION HOOK-UPS AND ATTACHMENTS WAS NOT COMPLETED; NONE OF THE SEVERAL STEPS WERE SIGNED OFF ALTHOUGH THE WORK WAS PERFORMED PRIOR TO CONDUCT OF THE TEST; AND (5) THE 13 FIELD CHANGES TO THE TEST WERE UNSIGNED AND WERE NOT APPROVED BY A LICENSED SRO PRIOR TO IMPLEMENTATION OF THE TEST.
(8019 INFRACTION)

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
NONE.				

AS REQUIRED BY TECHNICAL SPECIFICATION 6.4.A.4, DETAILED WRITTEN PROCEDURES WITH APPROPRIATE CHECK-OFF LISTS AND INSTRUCTIONS SHALL BE PROVIDED FOR ACTIONS TO BE TAKEN FOR SPECIFIC AND FORESEEN MALFUNCTIONS OF SYSTEMS OR COMPONENTS INCLUDING ALARMS, PRIMARY SYSTEM LEAKS AND ABNORMAL REACTIVITY CHANGES. CONTRARY TO THE ABOVE, DETAILED WRITTEN PROCEDURES WITH APPROPRIATE CHECK-OFF LISTS AND INSTRUCTIONS HAVE NOT BEEN PROVIDED AS OF APRIL 25, 1980, FOR ACTIONS TO BE TAKEN WHEN CERTAIN INSTALLED RADIATION MONITORING COMPONENTS MALFUNCTION. CONSEQUENTLY, WHEN THE UNIT 2 RADIATION MONITOR (RM-RMS-262) ON THE MANIPULATOR CRANE INTERMITTENTLY FAILED ON APRIL 21 AND 22, 1980, DURING UNIT 2 REFUELING, APPROPRIATE PROMPT CORRECTIVE ACTION WAS NOT TAKEN TO CORRECT THE MALFUNCTION WHICH WAS INDICATED BY LOSS OF THE GREEN FAIL/RESET LIGHT AND MR/HR METER INDICATION. SIMILARLY, PROCEDURES DO NOT EXIST FOR ACTIONS TO BE TAKEN WHEN FLOW OR FILTER FAULT ALARMS OCCUR ON RM-GW-101 AND RM-159, 160, 259 AND 260.
(8019 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

SYSTEM MODIFICATIONS AND REANALYSIS FOR SEISMIC SUPPORTS ARE CONTINUING.

FACILITY ITEMS (PLANS AND PROCEDURES):

ESTIMATE PLANT STARTUP~JULY 7, 1980.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

UNIT IS CURRENTLY SHUT DOWN CONDUCTING PREOPERATIONAL TESTING. THE UNIT HAS BEEN REFUELED.

LAST IE SITE INSPECTION DATE: JUNE 6, 1980

INSPECTION REPORT NO: 50-281/80-21

SUMMARY

INSPECTION SUMMARY

INSPECTION APRIL 15-17 (80-12): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 14 INSPECTOR-HOURS ON SITE IN THE AREAS OF STEAM GENERATOR SHIPMENT ACTIVITIES AND PREVIOUSLY IDENTIFIED ITEMS. OF THE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION APRIL 16-18 (80-13): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 12 INSPECTOR-HOURS ON SITE IN THE AREAS OF IE BULLETIN 79-14, "SEISMIC ANALYSIS FOR AS-BUILT SAFETY-RELATED PIPING SYSTEMS," AND IE BULLETIN 79-02, "PIPE SUPPORT BASE PLATE DESIGNS USING CONCRETE EXPANSION ANCHOR BOLTS." OF THE TWO AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN ONE AREA; ONE DEVIATION WAS FOUND IN ONE AREA (FAILURE TO MEET COMMITMENT FOR TIMELY PERFORMANCE OF IEB 79-02 INSPECTIONS).

INSPECTION APRIL 16-18 (80-15): THIS SPECIAL, ANNOUNCED INSPECTION INVOLVED 12 INSPECTOR-HOURS ON SITE IN THE AREA OF INSTALLED EQUIPMENT REVIEW WITH RESPECT TO THE IE BULLETIN 79-01B RESPONSE. OF THE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION APRIL 23 AND 25 (80-17): THIS SPECIAL, ANNOUNCED INSPECTION INVOLVED 10 INSPECTOR-HOURS ON SITE CONCERNING THE APRIL 14, 1980, SHIPMENT OF RADIOACTIVE WASTE FOR BURIAL INCLUDING SURVEYS, SHIPPING PAPER REQUIREMENTS, LABELING, PLACARDING AND RADIATION LEVELS. DETAILS I ADDRESSES INSPECTION ACTIVITIES AT THE WASTE DISPOSAL SITE. DETAILS II ADDRESSES INSPECTION ACTIVITIES AT SURRY POWER STATION. OF THE FIVE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN THREE AREAS; TWO ITEMS OF NONCOMPLIANCE WERE FOUND IN TWO AREAS (SEVERITY LEVEL II - VEHICLE SURFACE RADIATION LEVELS IN EXCESS OF REGULATORY LIMITS; AND SEVERITY LEVEL II - INADEQUATE SURVEY).

INSPECTION APRIL 29 - MAY 2 (80-18). THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 14 INSPECTOR-HOURS ON SITE IN THE AREAS OF IE BULLETIN 79-02, "PIPE SUPPORT BASE PLATE DESIGN USING CONCRETE EXPANSION ANCHOR BOLTS"; IE BULLETIN 79-14, "SEISMIC ANALYSIS FOR AS-BUILT SAFETY-RELATED PIPING SYSTEMS"; AND LICENSEE ACTIONS ON PREVIOUS INSPECTION FINDINGS. OF THE THREE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION APRIL 7 - MAY 2 (80-19): THIS ROUTINE INSPECTION BY THE RESIDENT INSPECTOR INVOLVED 50 INSPECTOR-HOURS ON SITE IN THE AREAS OF PLANT OPERATIONS AND OPERATING RECORDS, PLANT MODIFICATIONS, MAINTENANCE AND TESTING, FOLLOWUP ON PREVIOUSLY IDENTIFIED ITEMS, AND PLANT SECURITY. OF THE FOUR AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE FOUND IN TWO AREAS; TWO APPARENT ITEMS OF NONCOMPLIANCE WERE FOUND IN TWO AREAS (INFRACTION - INADEQUATE PROCEDURES FOR RADIATION MONITORING EQUIPMENT MALFUNCTIONS; AND INFRACTION - FAILURE TO FOLLOW TEST PROCEDURE FOR DC 77-9).

INSPECTION MAY 5-8 (80-20): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 13 INSPECTOR-HOURS ON SITE IN THE AREAS OF SCOPING THE SURRY 2 INTEGRATED STARTUP TEST PROGRAM AS TO THE INSPECTIONS AND TESTS REQUIRED PRIOR TO RETURNING THE UNIT TO OPERATION AFTER THE STEAM GENERATOR REPAIR PROGRAM, REVIEW THE COMPLETED RESULTS OF THE PRIMARY HYDROSTATIC TESTS AND REVIEW OF THE GENERAL ELECTRICAL, INSTRUMENT AND MECHANICAL TEST PROCEDURES. OF THE THREE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN TWO AREAS; ONE ITEM OF NONCOMPLIANCE WAS FOUND IN ONE AREA (INFRACTION - INADEQUATE TEST PROCEDURE).

REVIEWED BY OMPA LOEB:

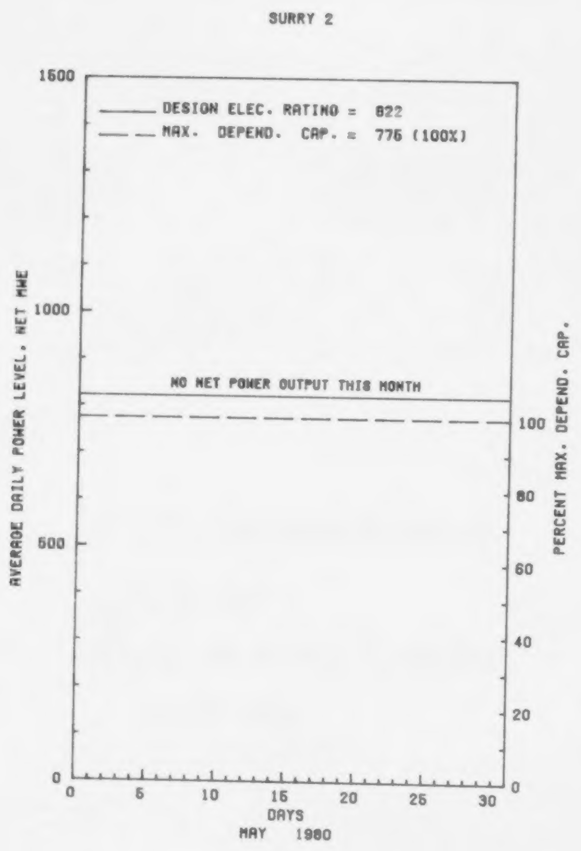
DATE: 7/3/80

REVIEWED BY NRR DOR: Don Neighbors

DATE: 7/7/80

SURRY 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-281
2. Utility Contact: O. J. Costello (804) 357-3184
3. Licensed Thermal Power (MWt): 2441
4. Nameplate Rating (Gross MWe): 942 x 0.9 = 848
5. Design Electrical Rating (Net MWe): 822
6. Maximum Dependable Capacity (Gross MWe): 811
7. Maximum Dependable Capacity (Net MWe): 775
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	3,647.0	62,111.0
12. Hours Reactor Was Critical	.0	.0	34,499.9
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	.0	.0	33,996.2
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	0	79,194,083
17. Gross Electrical Energy (MWH)	0	0	25,868,844
18. Net Electrical Energy (MWH)	0	0	24,536,605
19. Unit Service Factor	.0	.0	54.7
20. Unit Availability Factor	.0	.0	54.7
21. Unit Capacity Factor (MDC Net)	.0	.0	51.0
22. Unit Capacity Factor (DER Net)	.0	.0	48.1
23. Unit Forced Outage Rate	.0	.0	21.0

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 07/10/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-5	02/04/79	S	744.0	D	4				CONTINUATION OF SHUTDOWN FOR REFUELING & STEAM GENERATOR REPLACEMENT, WHICH BEGAN ON 02/04/79. WORK STILL UNDERWAY ON PIPING RESTRAINT MODIFICATIONS REQUIRED PRIOR TO UNIT RECOVERY.

SUMMARY

THE UNIT REMAINS SHUTDOWN FOR STEAM GENERATOR REPLACEMENT AND PIPING MODIFICATIONS.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

THREE MILE ISLAND 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Pennsylvania/Dauphin 10 miles SE of Harrisburg
2.	Electric Reliability Council	Mid-Atlantic Area Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2535
5.	Design Electrical Rating (MWE Net)	819
6.	Date of Initial Criticality	June 5, 1974
7.	Date of Commercial Operation	September 2, 1974
8.	Condenser Cooling Method	Closed
9.	Condenser Cooling Water Source	Susquehanna River

Utility & Contractor Information

10.	Licensee	Metropolitan Edison Company
11.	Corporate Address	P. O. Box 542 Reading, Pennsylvania
12.	Corporate Contact	R. C. Arnold, Sr. Vice-President 260 Cherry Hill Rd. Parsippany, NJ 07054
13.	Architect/Engineer	Gilbert Associates, Inc.
14.	Nuclear Steam Supply System	B&W
15.	Constructor	UE&C

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Section Chief	A. N. Fasano
19.	NRC-TMI Support Staff	FTS: 590-3950 D. Haverkamp (717) 782-3950
20.	Licensing Project Manager	D. C. DiIanni / H. Silver (X27435)
21.	Docket Number	50-289
22.	License Number/Date of Issue	DPR-50, 4/19/74
23.	Public Document Room	Government Publications Section State Library of Pennsylvania Education Building Commonwealth and Walnut Street Harrisburg, Pennsylvania 17126

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS:

The plant remains shutdown, by NRC Order, pending completion of modifications and other actions related to the TMI-2 accident.

LAST IE SITE INSPECTION DATE: 5/31/80

INSPECTION REPORT NO: 50-289/80-13

SUMMARY

INSPECTION SUMMARY

50-289/80-02 - Feb 1-Mar 16: Special inspection by NRC TMI Program Office Staff of: daily and selected shift activities. The inspection included daily (Monday-Friday) NRC TMI staff coverage and day shift coverage, with selected backshifts included, by NRC shift radiation specialists.

50-289/80-04 - Feb 5-9: Routine, unannounced inspection by four regionally-based inspectors (162 hrs) of work activities and records associated with the Unit 1 Restart Program, including: engineering design, quality assurance audits, electrical and instrumentation (Modifications Nos. RM-1, RM-4, RM-8, RM-10 and LM-8A), fire protection (Modification No. NM-40), installation of the hydrogen recombiner (Modification No. RM-12), and tendon surveillance. No items of noncompliance were identified.

50-289/80-06 - Mar 1-31: Routine, unannounced inspection by four regionally-based inspectors (224 hrs) of work activities and records associated with the Unit 1 Restart Program, including: engineering design, quality assurance audits, electrical heat shrink tubing/butt splicing (Modifications Nos. RM-1 and RM-4), fire protection (Modification No. NM-40), inservice non-destructive examination (ISI NDE) program, upgrade of the decay heat system (Modification No. RM-2), installation of the hydrogen recombiner (Modification No. RM-12), high pressure safety injection cross-connect piping (Modification No. RM-14), upgrade of the waste gas system (Modification No. LM-5), replacement of pipes cracked by intergranular stress corrosion (Modification No. NM-17), review of licensee event report 80-03, and tendon surveillance. No items of noncompliance were identified.

50-289/80-12 - Apr 23-25: Physical Barriers (Protected Area); Physical Barriers (Vital Area); Lighting; Access Control (Identification, Authorization, and Badging); Access Control (Search); Access Control (Vital Areas); Assessment Aids; Communications; Testing and Maintenance; Response; and Record and Reports. The inspection was initiated as a result of the licensee having reported a possible unauthorized entry into the owner controlled area by a unidentified vehicle. The inspection involved 22 inspector-hours onsite by one regional based NRC inspector. The inspection was begun during nonregular hours; eight (8) inspector hours were spent during off shift periods. No items of noncompliance were identified. It was determined that no unauthorized entry into the owner-controlled area had actually taken place and, therefore, there had been no real or potential threat to the physical protection of the site.

REPORTS RECEIVED FROM LICENSEE

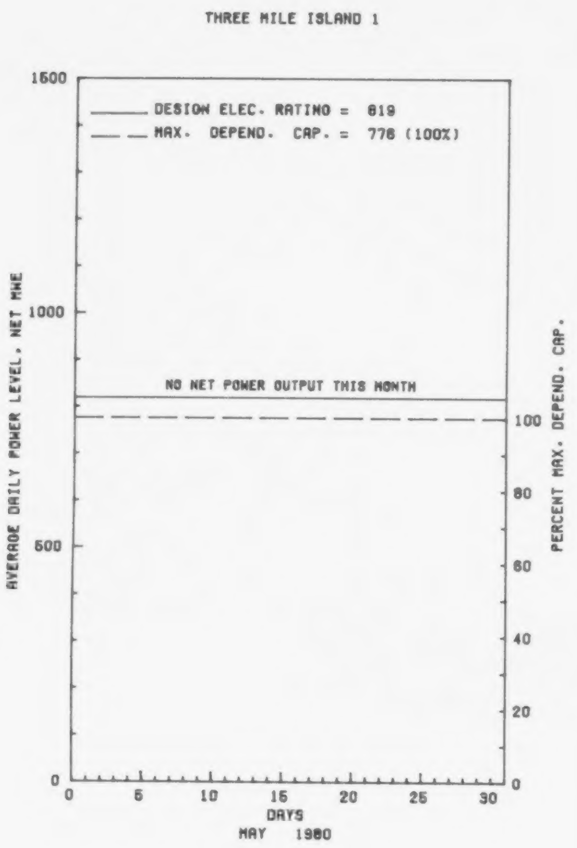
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-007/01T	04/23/80	05/08/80	Leak rate test system piping welds fail to meet ANSI B31.1.0 power piping code

REVIEWED BY OMPA LOEB: DATE: 7/3/80

REVIEWED BY NRR DOR: DATE: 7/7/80

THREE MILE ISLAND 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-289
2. Utility Contact: D. G. Mitchell (215) 921-6579
3. Licensed Thermal Power (MWt): 2535
4. Nameplate Rating (Gross MWe): 968 x 0.9 = 871
5. Design Electrical Rating (Net MWe): 819
6. Maximum Dependable Capacity (Gross MWe): 840
7. Maximum Dependable Capacity (Net MWe): 776
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	3,647.0	50,376.0
12. Hours Reactor Was Critical	.0	.0	31,731.8
13. Reactor Reserve Shutdown Hours	.0	.0	838.5
14. Hours Generator On-Line	.0	.0	31,180.9
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	0	76,531,071
17. Gross Electrical Energy (MWH)	0	0	25,484,330
18. Net Electrical Energy (MWH)	0	0	23,840,053
19. Unit Service Factor	.0	.0	61.9
20. Unit Availability Factor	.0	.0	61.9
21. Unit Capacity Factor (MDC Net)	.0	.0	61.0
22. Unit Capacity Factor (DER Net)	.0	.0	57.8
23. Unit Forced Outage Rate	100.0	100.0	27.4

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
1	02/17/79	F	744.0	D	4				REGULATORY RESTRAINT ORDER.

SUMMARY

THE UNIT REMAINS OFF LINE BECAUSE OF NRC RESTRAINT ORDER FOLLOWING ACCIDENT AT TMI-2.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

THREE MILE ISLAND 2

NRC POWER RESTRICTIONS: SEE "LICENSE STATUS"

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Pennsylvania/Dauphin 10 miles SE of Harrisburg
2.	Electric Reliability Council	Mid-Atlantic Area Council
3.	Type of Reactor	PWR
4.	Licenses Power Level (MWT)	2772
5.	Design Electrical Rating (MWE Net)	906
6.	Date of Initial Criticality	3/28/78
7.	Date of Commercial Operation	12/30/78
8.	Condenser Cooling Method	CLOSED
9.	Condenser Cooling Water Source	Susquehanna River

Utility & Contractor Information

10.	Licensee	Metropolitan Edison Company
11.	Corporate Address	P. O. Box 542 Reading, PA 19603
12.	Corporate Contact	R. C. Arnold, Sr. Vice-President 260 Cherry Hill Rd. Parsippany, NJ 07054
13.	Architect/Engineer	Burns & Roe
14.	Nuclear Steam Supply System	B&W
15.	Constructor	UE&C

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Section Chief	FTS: 590-3950 A. N. rasano (717) 782-3950
19.	IE Resident Inspector	FTS: 590-3950 R. Conte (717) 782-3950
20.	TMI Program Director	B. Snyder
21.	Docket Number	50-320
22.	License Number/Date of Issue	DPR-73/2; 2/8/78
23.	Public Document Room	Government Publications Section State Library of Pennsylvania Education Building Commonwealth & Walnut Streets Harrisburg, Pennsylvania 17126

INSPECTION STATUS

ENFORCEMENT STATUS

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEM AND COMPONENT PROBLEMS:

Recovery operations from 3/28/79 accident.

PLANT STATUS:

Decay heat is being removed by cyclic natural circulation through the 'A' Once Through Steam Generator and to reactor building ambient from GCS loops.

LAST IE SITE INSPECTION DATE: 5/31/80

INSPECTION REPORT NO: 50-320/80-10

SUMMARY

INSPECTION SUMMARY

50-320/80-02 - Feb 1-Mar 16: Special inspection by NRC TMI Program Office Staff of: licensee action on previous inspection findings; selected new and revised procedures; daily and selected shift activities; water leak from makeup pump instrumentation on February 11, 1980; reactor building atmosphere sample on February 12-13, 1980; and construction/plans for processed water storage tanks. The inspection included daily (Monday-Friday) NRC TMI staff coverage and day shift coverage, with selected backshifts included, by NRC shift radiation specialists. Five items of noncompliance were identified: (Failure to properly train vendor personnel; Failure of operators to properly implement a facility procedure; Failure to obtain an adequate airborne survey; Failure to properly post radiation area; and Failure to adequately survey for airborne tritium activity).

LICENSE STATUS

By Order for Modification of License dated July 20, 1979, the licensee's authority to operate the facility was suspended and the licensee's authority was limited to maintenance of the facility in the present shutdown cooling mode (44 F.R. 45271).

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-012/ 01L	04/13/80	05/13/80	Fuel handling exhaust flow less than required
80-013/ 01L	04/14/80	5/15/80	Erratic behavior of incore thermocouples G1, G5, E11, L13, O12 have previously failed

REVIEWED BY OMPA LOEB: RE Bobe **DATE:** 7/3/80

REVIEWED BY NRR DOR: D. S. Brinkman **DATE:** 7/9/80
(TMI PROGRAM OFFICE)

THREE MILE ISLAND 2

AVERAGE DAILY POWER LEVEL (MWe)

OPERATING STATUS

1. Docket Number: 50-320
2. Utility Data Prepared By: R. A. Lengel (215) 921-6581
3. Licensed Thermal Power (MWt): 2772
4. Nameplate Rating (Gross MWe): 1068 X 0.9 = 961
5. Design Electrical Rating (Net MWe): 906
6. Maximum Dependable Capacity (Gross MWe): 0
7. Maximum Dependable Capacity (Net MWe): 0
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons: None
9. Power Level To Which Restricted, If Any (Net MWe): None
10. Reasons For Restrictions, If Any:

This Month Yr

11. Hours In Reporting Period
12. Number Of Hours Reactor Was Critical
13. Reactor Reserve Shutdown Hours
14. Hours Generator On-Line
15. Unit Reserve Sh
16. Gro

(MWH)

19

20. U Availability Factor
21. Unit Capacity Factor (Using MDC Net) 1/
22. Unit Capacity Factor (Using DER Net) 1/
23. Unit Forced Outage Rate
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): None
25. If Shut Down At End Of Report Period, Estimated Date of Startup: N/A

NO DECISION ON FUTURE OPERATION
SHUTDOWN INDEFINITELY

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
SEE SUMMARY BELOW.									

SUMMARY

THE UNIT HAS REMAINED SHUTDOWN SINCE THE 03/28/79 ACCIDENT, WHICH CAUSED EXTENSIVE CORE DAMAGE.

AUTHORITY TO OPERATE FACILITY SUSPENDED BY NRC ORDER DATED JULY 20, 1979,
WHICH REQUIRES LICENSEE TO MAINTAIN FACILITY IN A SHUTDOWN CONDITION.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

TROJAN

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Oregon/Columbia 42 miles N of Portland
2.	Electric Reliability Council	Western Systems Coordinating Council
3.	Type of Reactor	PWR
4.	Licensed Power Level (MWT)	3411
5.	Design Electrical Rating (MWE Net)	1130
6.	Date of Initial Criticality	December 15, 1975
7.	Date of Commercial Operation	May 20, 1976
8.	Condenser Cooling Method	Natural draft cooling tower
9.	Condenser Cooling Water Source	Columbia River

Utility & Contractor Information

10.	Licensee	Portland General Electric Co.
11.	Corporate Address	121 S. W. Salmon St. Portland, Oregon 97204
12.	Corporate Contact	C. Goodwin, Jr., Asst. V.P.
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Multiple

Regulatory Information

16.	IE Region responsible	V
17.	IE Branch Chief	J. L. Crews
18.	IE Section Chief	D. M. Sternberg
19.	IE Resident Inspector	M. H. Malmros (503) 556-3751
20.	Licensing Project Manager	C. M. Trammell
21.	Docket Number	50-344
22.	License Number/Date of Issue	NPR-1, 11/21/75
23.	Public Document Room	Columbia County Courthouse Law Library Circuit Court Room St. Helens, Oregon 97501

INSPECTION STATUS

ENFORCEMENT STATUS

COMMUNICATION ROOM UNLOCKED AND UNATTENDED.
(8005 INFRACTION)

10 CFR 20.203(F)(2) REQUIRES THAT CONTAINERS OF RADIOACTIVE MATERIAL "BEAR A...LABEL IDENTIFYING THE CONTENTS..." INCLUDING "SUFFICIENT INFORMATION TO PERMIT INDIVIDUALS HANDLING OR USING THE CONTAINERS, OR WORKING IN THE VICINITY THEREOF, TO TAKE PRECAUTIONS TO AVOID OR MINIMIZE EXPOSURES." CONTRARY TO THIS REQUIREMENT, ON APRIL 17, 1980 THE LABELS ON SEVERAL SHIELDED CONTAINERS OF RADIOACTIVE WASTE STORED IN THE WASTE STORAGE AREA DID NOT PROVIDE ANY INFORMATION ON THE CONTENTS EXCEPT THE RADIATION CAUTION SYMBOL AND THE WORDS, "CAUTION RADIOACTIVE MATERIAL."
(8007 DEFICIENCY)

TECHNICAL SPECIFICATION 2.2.4.3, APPENDIX B, REQUIRES THAT THE PH OF DISCHARGES TO THE COLUMBIA RIVER FALL WITHIN THE RANGE OF 6.0 TO 9.0. CONTRARY TO THIS REQUIREMENT, DURING A PERIOD OF ABOUT 15 MINUTES ON OCTOBER 18, 1979, TWO SPIKES OCCURRED WHERE THE PLANT EFFLUENT BEING RELEASED TO THE COLUMBIA RIVER WAS AT A PH OF 9.05 AND 9.2, RESPECTIVELY.
(8007 DEFICIENCY)

INSPECTION STATUS

SYSTEMS AND COMPONENT PROBLEMS:

Resolution of questions regarding concrete block walls is required prior to startup.

FACILITY ITEMS (PLANS AND PROCEDURES):

None.

MANAGERIAL ITEMS:

None.

PLANT STATUS

The plant commenced a refueling and maintenance outage on April 11, 1980. Startup is scheduled for late June 1980.

LAST IE SITE INSPECTION DATE: 05/01-30/80+

INSPECTION REPORT NO: 50-344/80-11+

SUMMARY

INSPECTION SUMMARY:

Inspection on April 14-18, 1980 (Report No. 50-344/80-07)
Areas Inspected: Refueling preparations including organization; training procedures; advanced planning and preparation; personnel monitoring and exposure control; respiratory protection program; surveys; radiation work permits; radioactive and contaminated material control; tour of facility; followup of unusual event reported by licensee; licensee action on IE Bulletin 79-19; and licensee investigation of steam generator radiation safety program. The inspection involved 40 hours onsite by one NRC inspector.

Results: No items of noncompliance were found in 11 of the 13 areas inspected. Two items of noncompliance were identified (Deficiency - pH in plant effluent exceeded technical specification limit; and Deficiency - Several containers in the waste storage yard were not labeled in accordance with 10 CFR 20.203(f)(2)).

Inspection on April 8, 1980 (Report No. 50-344/80-08)
Inspection is continuing.

Inspection on April 21-24, 1980 (Report No. 50-344/80-09)
Areas Inspected: Routine, unannounced inspection of review and audit program; QA/QC administration program; procedures; procurement program; test and measuring equipment program; receipt, storage and handling; and independent inspection effort. The inspection involved 42 inspector hours onsite by two NRC inspectors.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

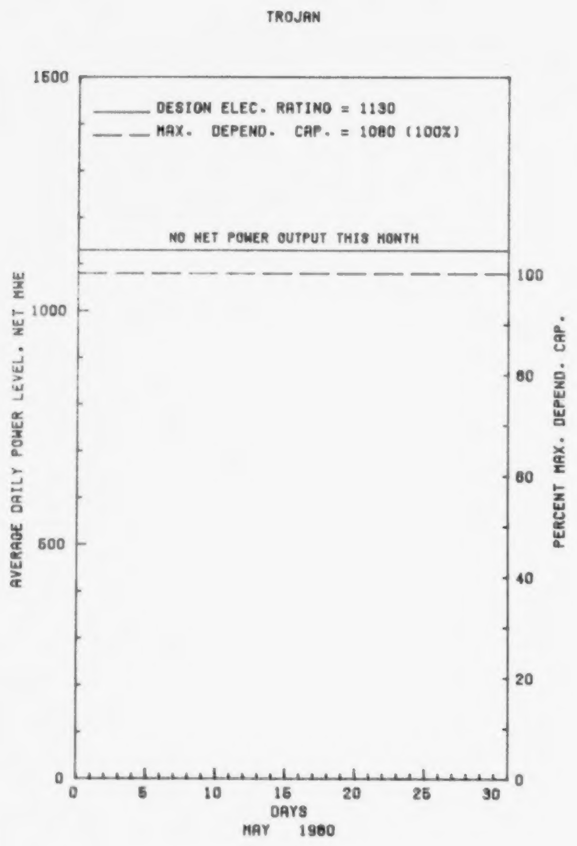
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-05/ 01T-0	04-11-80	04-22-80	Three of four MSIV's failed to close.
80-06/ 01T-0	04-25-80	05-08-80	One fuel rod failed in each of two fuel assemblies.

Results: No items of noncompliance or deviations were identified. One unresolved item concerning the neutron source range detectors was identified, whereupon the licensee committed to take corrective action.

Inspection on May 1-30, 1980 (Report No. 50-344/80-11)
Report being prepared; to be reported next month.

TROJAN

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-344
2. Utility Contact: G. G. Bair (503) 556-3713 x234
3. Licensed Thermal Power (MWt): 3411
4. Nameplate Rating (Gross MWe): 1280 x 0.95 = 1216
5. Design Electrical Rating (Net MWe): 1130
6. Maximum Dependable Capacity (Gross MWe): 1122
7. Maximum Dependable Capacity (Net MWe): 1080
8. If Changes Occur In Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	3,647.0	32,855.0
12. Hours Reactor Was Critical	.0	2,422.0	18,720.0
13. Reactor Reserve Shutdown Hours	.0	.0	2,171.8
14. Hours Generator On-Line	.0	2,416.8	18,044.6
15. Unit Reserve Shutdown Hours	.0	.0	1,508.7
16. Gross Thermal Energy (MWH)	0	6,839,434	55,735,044
17. Gross Electrical Energy (MWH)	0	2,203,790	18,189,205
18. Net Electrical Energy (MWH)	-2,442	2,092,533	17,125,852
19. Unit Service Factor	.0	66.3	54.9
20. Unit Availability Factor	.0	66.3	59.5
21. Unit Capacity Factor (MDC Net)	.0	53.1	48.3
22. Unit Capacity Factor (DER Net)	.0	50.8	46.1
23. Unit Forced Outage Rate	.0	1.0	27.7

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 06/22/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-04	04/11/80	S	744.0	C	4		RC	FUELXX	REFUELING CONTINUES.

SUMMARY

THE UNIT REMAINED SHUTDOWN THROUGHOUT THE REPORT PERIOD FOR REFUELING.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source

TURKEY POINT 3

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Florida/Dade 25 miles S of Miami
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2200
5.	Design Electrical Rating (MWE Net)	693
6.	Date of Initial Criticality	October 20, 1972
7.	Date of Commercial Operation	December 14, 1972
8.	Condenser Cooling Method	Recirculating Canal System
9.	Condenser Cooling Water Source	Atlantic Ocean

Utility & Contractor Information

10.	Licensee	Florida Power and Light Company
11.	Corporate Address	9250 West Flagler Street P. O. Box 013100 Miami, Florida 33101
12.	Corporate Contact	R. E. Uhrig, V.P. Nuclear Affairs
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. Lewis
18.	IE Section Chief	D. R. QUICK
19.	IE Resident Inspector	R. J. Vogt-Lowell (305) 245-7669
20.	Licensing Project Manager	M. Grotenhuis (301) 492-7128
21.	Docket Number	50-250
22.	License Number/Date of Issue	DPR-31, 7/19/72
23.	Public Document Room	Environmental and Urban Affairs Library Florida International University Miami, Florida 33199

INSPECTION STATUS

ENFORCEMENT STATUS

AS REQUIRED BY SECTION 6.9.2.B.3 OF THE TECHNICAL SPECIFICATIONS, OBSERVED INADEQUACIES IN THE IMPLEMENTATION OF ADMINSTRATIVE OR PROCEDURAL CONTROLS WHICH THREATEN TO CAUSE REDUCTION OF DEGREE OF REDUNDANCY PROVIDED IN REACTOR PROTECTION SYSTEMS OR ENGINEERED SAFETY FEATURE SYSTEMS SHALL BE REPORTED TO THE NRC. THE LACK OF PROPER CONTROL AND DOCUMENTATION OF THE BRACKET FABRICATION AND INSTALLATION PROCESS ON THE EMERGENCY DIESEL GENERATOR AS DISCUSSED IN THE JUNE 8, 1979, PLANT NUCLEAR SAFETY COMMITTEE MINUTES WAS NOT REPORTED TO THE NRC.
(7926 DEFICIENCY)

AS REQUIRED BY SECTION 6.8.1 OF THE TECHNICAL SPECIFICATIONS, WRITTEN PROCEDURES AND ADMINISTRATIVE POLICIES THAT MEET OR EXCEED THE REQUIREMENTS AND RECOMMENDATIONS OF SECTION 5.1 AND 5.3 OF ANSI N18.7-1972 AND APPENDIX A OF USNRC REGULATORY GUIDE 1.33 SHALL BE IMPLEMENTED. THE FOLLOWING TWO EXAMPLES REPRESENT FAILURE TO IMPLEMENT EXISTING PROCEDURES: (1) REPLACEMENT OF THE ORIGINAL CONTROLLER FOR SPRAY VALVE PCV-3-455 B WITH A DIFFERENT TYPE CONTROLLER WHICH WAS IN USE DURING THE INSPECTION PERIOD WAS NOT PERFORMED UNDER THE GUIDELINES OF A.P. 190.15, "PLANT CHANGES AND MODIFICATIONS"; (2) NO TEMPORARY PROCEDURE HAD BEEN PROMULGATED TO INSURE CONTINUOUS UNDERSTANDING BETWEEN SHIFTS OF THE OPERATIONAL CONTROL OVER THE NEW SPRAY VALVE CONTROLLER AS REQUIRED BY A.P. 103.6, "TEMPORARY PROCEDURES".
(7926 INFRACTION)

AS REQUIRED BY TECHNICAL SPECIFICATION 6.8.1 WRITTEN PROCEDURES AND ADMINISTRATIVE POLICIES SHALL BE ESTABLISHED, IMPLEMENTED AND MAINTAINED THAT MEET OR EXCEED THE REQUIREMENTS OF APPENDIX "A" OF THE USNRC REGULATORY GUIDE 1.33. REGULATORY GUIDE 1.33, APPENDIX A, STATES IN PART THAT CHEMICAL AND RADIOCHEMICAL CONTROL PROCEDURES SHOULD BE WRITTEN TO PRESCRIBE THE NATURE AND FREQUENCY OF SAMPLING AND THAT THESE PROCEDURES SHOULD SPECIFY LABORATORY INSTRUCTIONS AND CALIBRATION OF LABORATORY EQUIPMENT. ON MARCH 26, 1980, THERE WERE NO WRITTEN PROCEDURES FOR EFFICIENCY CALIBRATION OF THE GE(LI) GAMMA SPECTROSCOPY SYSTEM AND THE LIQUID SCINTILLATION COUNTER. DURING THE PERIOD FROM MAY 1979 TO MARCH 1980, PROCEDURE NC-11, "GAS FLOW PROPORTIONAL COUNTER DAILY QUALITY CONTROL CHECK", WAS NOT FOLLOWED IN THAT RECOUNTS WERE NOT MADE WHEN THE DAILY CHECKS DID NOT MEET THE ACCEPTANCE CRITERIA.
(8015 DEFICIENCY)

BASED ON THE RESULTS OF THE NRC INSPECTION CONDUCTED ON FEBRUARY 1-29, 1980, CERTAIN OF YOUR ACTIVITIES APPEAR TO DEVIATE FROM YOUR COMMITMENTS TO THE COMMISSION. AS INDICATED IN PARAGRAPH 5 OF IE INSPECTION REPORT 50-250-251/79-26, A COMMITMENT WAS MADE TO HAVE AVAILABLE BY OCTOBER 1979, FOR USE IN FINAL, APPROVED FORM, PROCEDURES COVERING THE FOLLOWING: (1) AUXILIARY FEEDWATER PUMP TURBINE OVERHAUL; (2) AUXILIARY FEEDWATER PUMP TURBINE THROTTLE TRIP VALVE OVERHAUL; (3) AUXILIARY FEEDWATER PUMP OVERHAUL AND MAINTENANCE. SUCH PROCEDURES HAD NOT BEEN MADE AVAILABLE FOR USE IN FINAL APPROVED FORM AS OF FEBRUARY 28, 1980.
(8010 INFRACTION)

AS INDICATED IN PARAGRAPH 5 OF IE INSPECTION REPORT 50-250-251/79-26, A COMMITMENT WAS MADE TO HAVE AVAILABLE BY OCTOBER 1979, FOR USE IN FINAL, APPROVED FORM, PROCEDURES COVERING THE FOLLOWING: (1) AUXILIARY FEEDWATER PUMP TURBINE OVERHAUL; (2) AUXILIARY FEEDWATER PUMP TURBINE THROTTLE TRIP VALVE OVERHAUL; (3) AUXILIARY FEEDWATER PUMP OVERHAUL AND MAINTENANCE. SUCH PROCEDURES HAD NOT BEEN AVAILABLE FOR USE IN FINAL APPROVED FORM AS OF FEBRUARY 28, 1980.
(8010 DEVIATION)

AS INDICATED IN TABLE 9-1 PARAGRAPH C.2.(C) OF YOUR FEBRUARY 25, 1977, FIRE HAZARDS ANALYSIS, THE FIRE PUMP INSTALLATION COMPLIES WITH NFPA-20. AUDIBLE OR VISUAL ALARMS AT A POINT OF CONSTANT ATTENDANCE WHICH WOULD ALARM UPON LOSS OF POWER ON THE LINE SIDE OF THE MOTOR STARTER WERE NOT PROVIDED AS REQUIRED BY NFPA-20, IN THE INSTALLATION OF THE FIRE PUMPS.
(8015 DEVIATION)

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-006/ 03L-0	03/16/80	04/15/80	FAILURE TO PERFORM SURVEILLANCE OF VALVE POSITION FOR CERTAIN FIRE MAIN VALVES.

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

NORMAL OPERATION AT 100% POWER.

LAST IE SITE INSPECTION DATE: JUNE 4-6, 1980

INSPECTION REPORT NO: 50-250/80-19

SUMMARY

INSPECTION SUMMARY

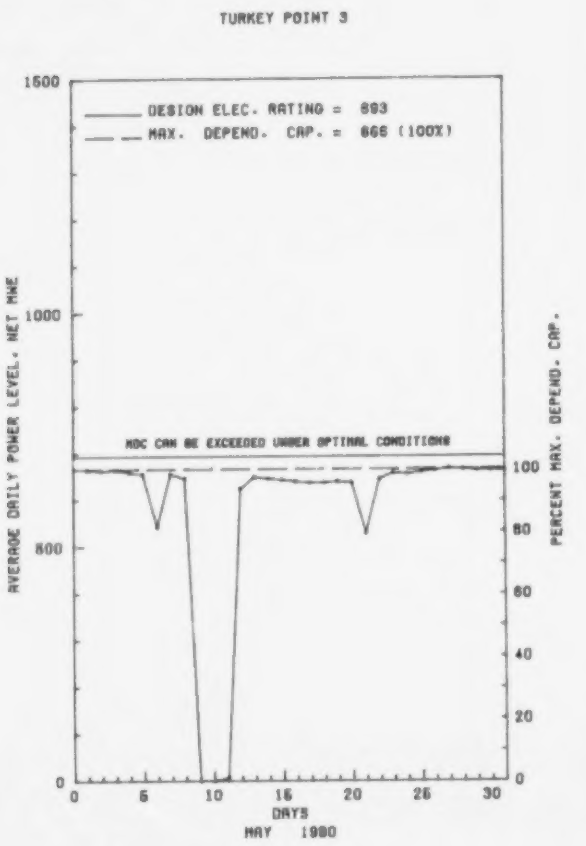
INSPECTION APRIL 1-30 (80-16): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 71 INSPECTOR-HOURS ON SITE IN THE AREAS OF FOLLOWUP OF PREVIOUS INSPECTION FINDINGS; PLANT OPERATIONS; FOLLOWUP OF SIGNIFICANT EVENT NOTIFICATIONS; MAINTENANCE OBSERVATION; LICENSEE ACTION ON IE BULLETINS. OF THE FIVE AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE WERE IDENTIFIED IN FOUR AREAS; ONE APPARENT ITEM OF NONCOMPLIANCE WAS FOUND IN ONE AREA (INFRACTION - FAILURE TO TRACK IMPLEMENTATION OF CORRECTIVE ACTION; ONE APPARENT DEVIATION WAS FOUND IN ONE AREA (DEVIATION - FAILURE TO PERFORM CORRECTIVE ACTION BY DATE STATED IN LICENSEE'S RESPONSE TO AN ITEM OF NONCOMPLIANCE).

REVIEWED BY OMPA LOEB: L. Tomson **DATE:** 7 3 80

REVIEWED BY NRR DOR: M Grotenhuis **DATE:** 7-7-80

TURKEY POINT 3

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-250
2. Utility Contact: V. T. Chilson (305) 552-3824
3. Licensed Thermal Power (MWt): 2200
4. Nameplate Rating (Gross MWe): 894 x 0.85 = 760
5. Design Electrical Rating (Net MWe): 693
6. Maximum Dependable Capacity (Gross MWe): 700
7. Maximum Dependable Capacity (Net MWe): 666

ITEM 21 CALCULATED BY USING A WEIGHTED AVERAGE

8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	3,647.0	65,648.6
12. Hours Reactor Was Critical	677.2	2,772.4	50,041.8
13. Reactor Reserve Shutdown Hours	.0	.0	213.3
14. Hours Generator On-Line	671.0	2,617.7	48,387.9
15. Unit Reserve Shutdown Hours	.0	.0	121.8
16. Gross Thermal Energy (MWH)	1,458,855	5,618,082	97,086,716
17. Gross Electrical Energy (MWH)	456,540	1,783,585	30,864,585
18. Net Electrical Energy (MWH)	432,468	1,687,317	29,209,411
19. Unit Service Factor	90.2	71.8	73.6
20. Unit Availability Factor	90.2	71.8	73.8
21. Unit Capacity Factor (MDC Net)	87.3	69.5	67.3
22. Unit Capacity Factor (DER Net)	83.9	66.8	64.2
23. Unit Forced Outage Rate	.0	1.8	2.7

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
STEAM GEN TUBE INSPEC PROG - OCT. 5 - NOV. 1, 1980
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
11	05/06/80	F	2.9	A	3		ED	CKTBRK	UNIT NO. 3 WAS TRIPPED BY STEAM GENERATOR NO. 3C LEVEL PROTECTION SYSTEM DURING A TRANSIENT CONDITION CAUSED BY THE LOSS OF POWER SUPPLY TO VITAL INSTRUMENT BUS NOS. 3B AND 4A DUE TO THE MALFUNCTION OF BUS TIE CIRCUIT BREAKER NO. 3B-20 CORRECTIVE ACTIONS INCLUDED EXERCISING THE CIRCUIT BREAKER.
12	05/09/80	S	67.6	F	1		CB	SUPORT	UNIT WAS REMOVED FROM SERVICE TO MODIFY SUPPORTS AND RESTRAINTS ON PIPING LOCATED INSIDE CONTAINMENT.
13	05/21/80	F	2.5	A	3		ED	BATTRY	UNIT WAS TRIPPED BY STEAM GENERATOR NO. 3B LEVEL PROTECTION DURING A TRANSIENT CONDITION CAUSED BY THE LOSS OF POWER SUPPLY TO VITAL INSTRUMENT BUS NO. 3A DUE TO A MALFUNCTION OF INVERTER NO. 3A. CORRECTIVE ACTIONS INCLUDED PLACING SPARE INVERTER NO. AS IN SERVICE.

SUMMARY

THE UNIT OPERATED NEAR FULL POWER ALL MONTH EXCEPT FOR OUTAGES ON 05/06/80 AND 05/14/80 CAUSED BY THE STEAM GENERATOR TRIPS DUE TO INSTRUMENT POWER LOSS AND ON 05/09/80 FOR CONTAINMENT PIPING SUPPORTS.

LEGEND

[1] Type:
F-Forced
S-Scheduled

[2] Reason:
A-Equipment Failure
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error
H-Other

[3] Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Continuations
5-Load Reduction
9-Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

TURKEY POINT 4

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Florida/Dade 25 miles S of Miami
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2200
5.	Design Electrical Rating (MWE Net)	693
6.	Date of Initial Criticality	June 11, 1973
7.	Date of Commercial Operation	September 7, 1973
8.	Condenser Cooling Method	Recirculating Canal System
9.	Condenser Cooling Water Source	Atlantic Ocean

Utility & Contractor Information

10.	Licensee	Florida Power and Light Company
11.	Corporate Address	9250 West Flagler Street P. O. Box 013100 Miami, Florida 33101
12.	Corporate Contact	R. E. Uhrig, V. P. Nuclear Affairs
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	II
	IE Branch Chief	R. C. Lewis
18.	IE Section Chief	D. R. QUICK
19.	IE Resident Inspector	R. J. Vogt-Lowell (305) 245-7669
20.	Licensing Project Manager	M. Grotenhuis (301) 492-7128
21.	Docket Number	50-251
22.	License Number/Date of Issue	DPR-41, 4/10/73
23.	Public Document Room	Environmental and Urban Affairs Library Florida International University Miami, Florida 33199

INSPECTION STATUS

ENFORCEMENT STATUS

AS REQUIRED BY SECTION 6.9.2.B.3 OF THE TECHNICAL SPECIFICATIONS, OBSERVED INADEQUACIES IN THE IMPLEMENTATION OF ADMINISTRATIVE OR PROCEDURAL CONTROLS WHICH THREATEN TO CAUSE REDUCTION OF DEGREE OF REDUNDANCY PROVIDED IN REACTOR PROTECTION SYSTEMS OR ENGINEERED SAFETY FEATURE SYSTEMS SHALL BE REPORTED TO THE NRC. THE LACK OF PROPER CONTROL AND DOCUMENTATION OF THE BRACKET FABRICATION AND INSTALLATION PROCESS ON THE EMERGENCY DIESEL GENERATOR AS DISCUSSED IN THE JUNE 8, 1979, PLANT NUCLEAR SAFETY COMMITTEE MINUTES WAS NOT REPORTED TO THE NRC.
(7920 DEFICIENCY)

AS REQUIRED BY TECHNICAL SPECIFICATION 6.8.1 WRITTEN PROCEDURES AND ADMINISTRATIVE POLICIES SHALL BE ESTABLISHED, IMPLEMENTED AND MAINTAINED THAT MEET OR EXCEED THE REQUIREMENTS OF APPENDIX "A" OF THE USNRC REGULATORY GUIDE 1.33. REGULATORY GUIDE 1.33, APPENDIX A, STATES IN PART THAT CHEMICAL AND RADIOCHEMICAL CONTROL PROCEDURES SHOULD BE WRITTEN TO PRESCRIBE THE NATURE AND FREQUENCY OF SAMPLING AND THAT THESE PROCEDURES SHOULD SPECIFY LABORATORY INSTRUCTIONS AND CALIBRATION OF LABORATORY EQUIPMENT. ON MARCH 26, 1980, THERE WERE NO WRITTEN PROCEDURES FOR EFFICIENCY CALIBRATION OF THE [illegible] GAMMA SPECTROSCOPY SYSTEM AND THE LIQUID SCINTILLATION COUNTER. DURING THE PERIOD FROM MAY 1979 TO MARCH 1980, PROCEDURE NC-13, "GAS PROPORTIONAL COUNTER DAILY QUALITY CONTROL CHECK", WAS NOT FOLLOWED IN THAT RECOUNTS WERE NOT MADE WHEN THE DAILY CHECKS DID NOT MEET THE ACCEPTANCE CRITERIA.
(8008 DEFICIENCY)

AS INDICATED IN TABLE 9-1 PARAGRAPH C.2.(C) OF YOUR FEBRUARY 25, 1977, FIRE HAZARDS ANALYSIS, THE FIRE PUMP INSTALLATION COMPLIES WITH NFPA-20. AUDIBLE OR VISUAL ALARMS AT A POINT OF CONSTANT ATTENDAANDE WHICH WOULD ALARM UPON LOSS OF POWER ON THE LINE SIDE OF THE MOTOR STARTER WERE NOT PROVIDED AS AREQUIRE BY NFPA 20, IN THE INSTALLATION OF THE FIRE PUMPS.
(8013 DEVIATION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

UNIT SHUT DOWN FOR STEAM GENERATOR TUBE INSPECTION LOW PRESSURE TURBINE ROTOR REPLACEMENT, AND CRACKS IN STEAM GENERATOR FEEDWATER NOZZLE-TO-PIPE REDUCERS.

LAST IE SITE INSPECTION DATE: JUNE 4-6, 1980

INSPECTION REPORT NO: 50-251/80-20

SUMMARY

INSPECTION SUMMARY

INSPECTION APRIL 1-30 (80-14): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 71 INSPECTOR-HOURS ON SITE IN THE AREAS OF FOLLOWUP OF PREVIOUS INSPECTION FINDINGS; PLANT OPERATIONS; FOLLOWUP OF SIGNIFICANT EVENT NOTIFICATIONS; MAINTENANCE OBSERVATION; LICENSEE ACTION ON IE BULLETINS. OF THE FIVE AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE WERE IDENTIFIED IN FOUR AREAS; ONE APPARENT ITEM OF NONCOMPLIANCE WAS FOUND IN ONE AREA (INFRACTION - FAILURE TO TRACK IMPLEMENTATION OF CORRECTIVE ACTION; ONE APPARENT DEVIATION WAS FOUND IN ONE AREA. DEVIATION - FAILURE TO PERFORM CORRECTIVE ACTION BY DATE STATED IN LICENSEE'S RESPONSE TO AN ITEM OF NONCOMPLIANCE).

INSPECTION MAY 15-16 (80-16): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 10 INSPECTOR-HOURS ON SITE IN THE AREA OF STEAM GENERATOR FEEDWATER LINE RADIOGRAPHY (RT) PER IE BULLETIN 79-13. NO ITEMS OF NONCOMPLIANCE OR DEVIATION WERE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

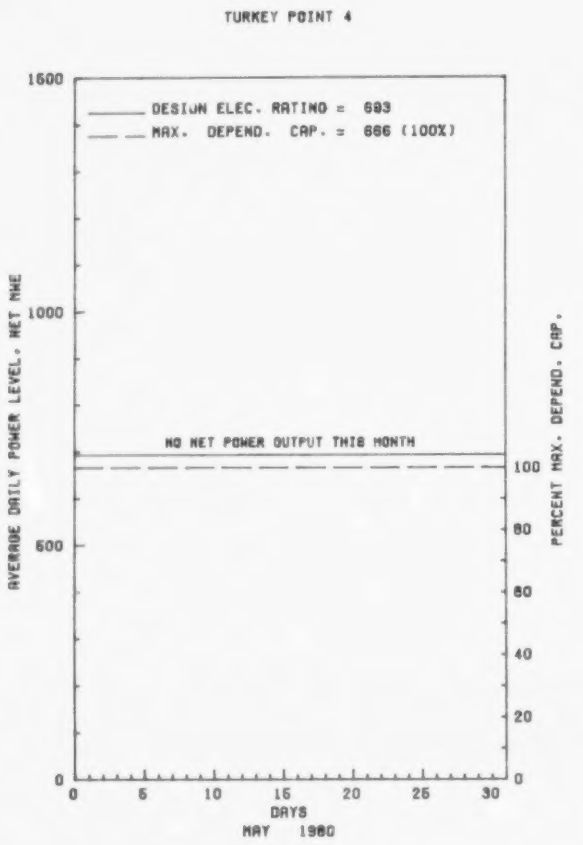
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-005/ 03L-0	03/03/80	04/30/80	PROCESS RADIATION MONITORS OUT OF SERVICE.
80-006/ 03L-0	04/07/80	05/07/80	AUXILIARY FEEDWATER PUMP FAILED TO DELIVER DESIGNED FLOWRATE DURING PERFORMANCE TEST.

REVIEWED BY OMPA LOEB: L. Tomzon DATE: 7. 3. 80

REVIEWED BY NRR DOR: MGrotenhuis DATE: 7-7-80

TURKEY POINT 4

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-251
2. Utility Contact: V. T. Chilson (305) 552-3824
3. Licensed Thermal Power (MWt): 2200
4. Nameplate Rating (Gross MWe): 894 x 0.85 = 760
5. Design Elect cal Rating (Net MWe): 693
6. Maximum Dependable Capacity (Gross MWe): 700
7. Maximum Dependable Capacity (Net MWe): 666
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

ITEM 21 CALCULATED BY USING A WEIGHTED AVERAGE.

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	3,647.0	59,376.0
12. Hours Reactor Was Critical	.0	2,666.0	43,493.8
13. Reactor Reserve Shutdown Hours	.0	12.3	166.6
14. Hours Generator On-Line	.0	2,616.1	41,798.6
15. Unit Reserve Shutdown Hours	.0	12.3	31.2
16. Gross Thermal Energy (MWH)	0	5,444,694	86,883,634
17. Gross Electrical Energy (MWH)	0	1,780,840	27,484,318
18. Net Electrical Energy (MWH)	-1,455	1,687,714	26,217,710
19. Unit Service Factor	.0	71.7	70.4
20. Unit Availability Factor	.0	72.1	70.4
21. Unit Capacity Factor (MDC Net)	.0	69.5	66.6
22. Unit Capacity Factor (DER Net)	.0	66.8	63.7
23. Unit Forced Outage Rate	.0	.4	3.1

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 06/17/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
13	04/26/80	S	744.0	B	4		HB	HTEXCH	UNIT WAS REMOVED FROM SERVICE TO PERFORM SCHEDULED MAINTENANCE AND INSPECTIONS, INCLUDING STEAM GENERATOR TUBE INSPECTION PROGRAM, AND TURBINE L.P. ROTOR CHANGEOUT.

SUMMARY

THE UNIT CONTINUED AN INSPECTION AND MAINTENANCE OUTAGE BEGUN IN APRIL THROUGHOUT MAY.

LEGEND

[1] Type:
F–Forced
S–Scheduled

[2] Reason:
A–Equipment Failure
B–Maintenance or Test
C–Refueling
D–Regulatory Restriction
E–Operator Training & License Examination
F–Administrative
G–Operational Error
H–Other

[3] Method:
1–Manual
2–Manual Scram
3–Automatic Scram
4–Continuations
5–Load Reduction
9–Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

VERMONT YANKEE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Vermont/Windham 5 miles S of Brattleboro
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	1593
5.	Design Electrical Rating (MWE Net)	514
6.	Date of Initial Criticality	March 24, 1972
7.	Date of Commercial Operation	November 30, 1972
8.	Condenser Cooling Method	Closed Cycle Cooling Towers
9.	Condenser Cooling Water Source	Connecticut River

Utility & Contractor Information

10.	Licensee	Vermont Yankee Nuclear Power Corp.
11.	Corporate Address	20 Turnpike Road Westboro, Massachusetts 01581
12.	Corporate Contact	D. W. Edwards Assistant to the Vice President
13.	Architect/Engineer	Ebasco
14.	Nuclear Steam Supply System	GE
15.	Constructor	Ebasco

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	IE Section Chief	T. T. MARTIN
19.	IE Resident Inspector	W. J. RAYMOND (802) 257-4319
20.	Licensing Project Manager	V. L. Rooney
21.	Docket Number	50-271
22.	License Number/Date of Issue	DPR-28, 3/21/72
23.	Public Document Room	Brooks Memorial Library 224 Main Street Brattleboro, Vermont 05301

INSPECTION STATUS

ENFORCEMENT STATUS

10 CFR 71.5(A) "TRANSPORTATION OF LICENSED MATERIAL," REQUIRES THAT NRC LICENSEES COMPLY WITH THE APPLICABLE PACKAGING AND TRANSPORTATION REQUIREMENTS OF THE U.S. DEPARTMENT OF TRANSPORTATION IN 49 CFR PARTS 170-189. 49 CFR 173.393(I) REQUIRES THAT ALL RADIOACTIVE MATERIALS MUST BE PACKAGED IN SUITABLE PACKAGING (SHIELDED, IF NECESSARY) SO THAT AT ANY TIME DURING NORMAL CONDITIONS INCIDENT TO TRANSPORTATION, THE RADIATION DOSE RATE DOES NOT EXCEED 200 MILLIREM PER HOUR AT ANY POINT ON THE EXTERNAL SURFACE OF THE PACKAGE. CONTRARY TO THE ABOVE, ON FEBRUARY 7, 1980, THE LICENSEE, AS CONSIGNOR, DELIVERED A PACKAGE OF LOW SPECIFIC ACTIVITY (LSA) RADIOACTIVE MATERIAL TO A CARRIER FOR TRANSPORT WITH A RADIATION DOSE RATE EXCEEDING 200 MILLIREM PER HOUR, AS EVIDENCED BY A MEASURED LEVEL OF 300 MILLIREM PER HOUR FROM AN UNDERNEATH EXTERNAL SURFACE OF THE VEHICLE UPON ITS ARRIVAL AT THE RICHLAND, WASHINGTON, WASTE BURIAL SITE ON FEBRUARY 14, 1980. (8003 VIOLATION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

On May 16, 1980, operators noted a temperature drop on the exit of the AOG Recombiner. Preliminary investigation indicated hydrogen recombination upstream of the Recombiner. System temperatures returned to normal on May 17, 1980 following performance of turbine surveillance which bypassed steam into condenser. Licensee Event Report 80-19/1P was issued, investigation is continuing.

FACILITY ITEMS (PLANS AND PROCEDURES):

Unit refueling outage is scheduled for September 1980.

MANAGERIAL ITEMS:

Management meeting between licensee and Region I staff scheduled for June 27, 1980, at the Vermont Yankee site.

PLANT STATUS:

Routine operations at rated power with plant cooling towers in service. Offgas and stack release rates are 3360 mc/sec and less than 100 mc/sec, respectively.

LAST IE SITE INSPECTION DATE: 5/23/80

INSPECTION REPORT NO: 50-271/80-07

SUMMARY

INSPECTION SUMMARY

50-271/80-03 - Feb 21: Special unannounced inspection by two regionally-based inspectors (8 hrs) of the circumstances surrounding a shipment of low specific activity (LSA) radioactive waste on a tractor-trailer which had radiation levels above the DOT limits given in 49 CFR 173.393(j), when the tractor-trailer arrived at the Nuclear Engineering Company (NECO), Richland, Washington, burial site. Another region-based inspector discussed the reporting of significant events with the licensee. One item of noncompliance was identified: (Failure to package a low specific activity (LSA) waste package so that the radiation levels from the surface of the package were within the limits given in 49 CFR 173.393(i)).

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

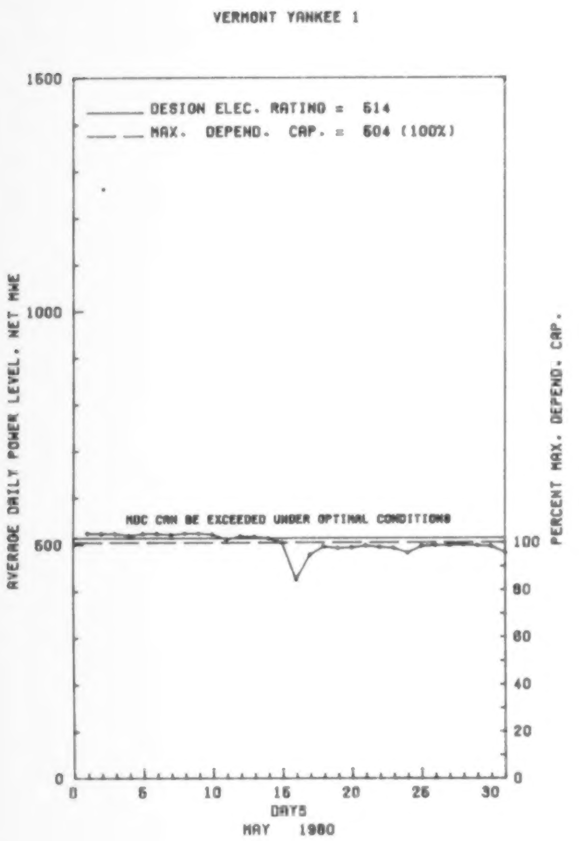
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-014/03L	04/02/80	05/02/80	UPS-A frequency drift during reactor startup
80-015/03L	04/01/80	05/01/80	Recirc pump dish vlv inop due to loose motor op pinion gear
80-016/03L	04/15/80	05/15/80	TS required quarterly calibration of main steam line radiation monitors in advertently missed in 12/79

REVIEWED BY OMPA LOEB: L. [illegible] **DATE:** 7/3/80

REVIEWED BY NRR DOR: Rooney **DATE:** 7/2/80

VERMONT YANKEE

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-271
2. Utility Contact: F.C. BEERS (617) 366-9011
3. Licensed Thermal Power (MWt): 1593
4. Nameplate Rating (Gross MWe): 626 x 0.9 = 563
5. Design Electrical Rating (Net MWe): 514
6. Maximum Dependable Capacity (Gross MWe): 535
7. Maximum Dependable Capacity (Net MWe): 504
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	3,647.0	67,441.8
12. Hours Reactor Was Critical	744.0	3,501.3	54,659.9
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	744.0	3,458.6	52,786.4
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,173,199	5,339,446	74,564,057
17. Gross Electrical Energy (MWH)	395,401	1,823,134	24,792,804
18. Net Electrical Energy (MWH)	374,298	1,744,461	23,512,826
19. Unit Service Factor	100.0	94.8	78.3
20. Unit Availability Factor	100.0	94.8	78.3
21. Unit Capacity Factor (MDC Net)	99.8	94.9	69.2
22. Unit Capacity Factor (DER Net)	97.9	93.1	67.8
23. Unit Forced Outage Rate	.0	.8	6.1

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUELING SCHEDULED ON 09/26/80, & WILL LAST FOR 8 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-12	05/11/80	S	.0	H	5		RA	CONROD	MAIN TURBINE AND CONTROL ROD SURVEILLANCE.
80-13	05/16/80	S	.0	H	5		MB	XXXXXX	POWER REDUCED TO INVESTIGATE AOG PROCESSING PROBLEM.

SUMMARY

THE UNIT RAN NEAR FULL POWER EXCEPT FOR REDUCTIONS ON 05/11/80 FOR SURVEILLANCE TESTING, AND ON 05/16/80 FOR AOG PROCESSING PROBLEMS. ON 05/15/80 THE PLANT CHANGED TO CLOSED CYCLE COOLING MODE, REDUCING THE MDC.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

YANKEE ROWE

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Massachusetts/Franklin 25 miles NE of Pittsfield
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	600
5.	Design Electrical Rating (MWE Net)	175
6.	Date of Initial Criticality	August 19, 1960
7.	Date of Commercial Operation	July 1961
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Deerfield River

Utility & Contractor Information

10.	Licensee	Yankee Atomic Electric Company
11.	Corporate Address	20 Turnpike Road Westboro, Massachusetts 01581
12.	Corporate Contact	D. W. Edwards, Asst. to V.P.
13.	Architect/Engineer	Stone & Webster
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Section Chief	T. T. MARTIN
19.	IE Resident Inspector	T. FOLEY (413)424-5242
20.	Licensing Project Manager	A. Burger
21.	Docket Number	50-29
22.	License Number/Date of Issue	DPR-3, 12/24/63
23.	Public Document Room	Greenfield Community College 1 College Drive Greenfield, Massachusetts 01301

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

FACILITY ITEMS (PLANS AND PROCEDURES):

Licensee has commenced outage maintenance and modifications on fire protection systems, containment isolation systems, steam generator seismic supports and emergency feedwater system.

PLANT STATUS:

Plant remains shutdown and cooled down. LP turbine repairs will continue thru September, 1980.

LAST IE SITE INSPECTION DATE: 5/27/80

INSPECTION REPORT NO: 50-29/80-10

SUMMARY

INSPECTION SUMMARY

50-29/80-02 - Jan 28-Feb 5: Routine, unannounced inspection by two regionally-based inspectors (55 hrs) of plant operations, including a tour of accessible areas, a review of shift logs, operating and shutdown records; observing maintenance activities; review and onsite followup of Licensee Event Reports (LER's); followup of selected IE Bulletins; and, a review of Category "A" Short Term Lessons Learned Items. Two items of noncompliance were identified: (Failure to maintain a required Emergency Core Cooling System operable during operation; and Failure to follow procedures).

50-29/80-03 - Feb 15-19: Special, unannounced inspection by a regionally-based inspector (33 hrs) of the circumstances surrounding the failure of Low Pressure Turbine. No items of noncompliance were identified.

50-29/80-04 - Feb 27: Management meeting held at NRC request to discuss details related to the establishment of an onsite office for the purpose of placing a full-time NRC inspector at the facility. All parties concurred on administrative and operational considerations needed to establish an onsite NRC inspector.

50-29/80-07 - Mar 17-24: Routine, unannounced inspection by two regionally-based inspectors (46 hrs) of licensee emergency procedures for small-break loss of coolant accidents; review of Licensee Event Reports (LERs) and Monthly Operating Reports. No items of noncompliance were identified.

REPORTS RECEIVED FROM LICENSEE

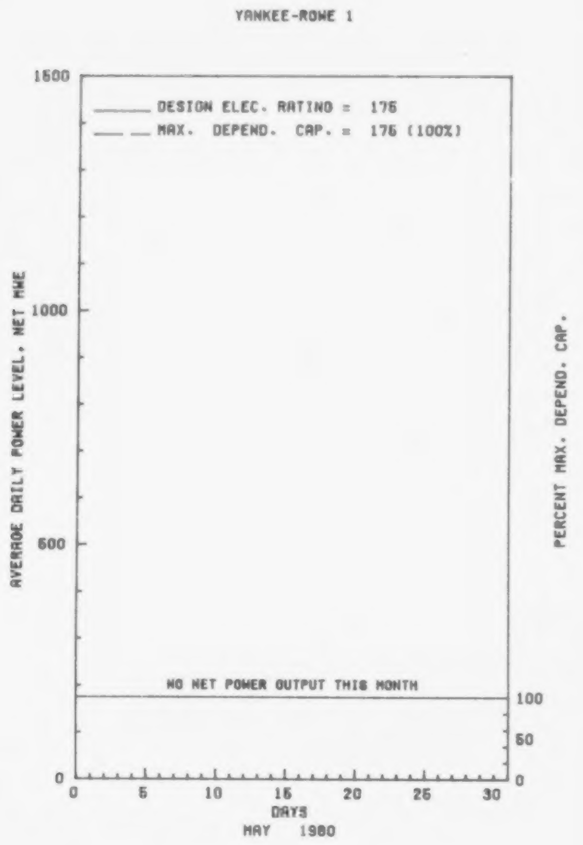
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-010/ 03L	04/06/80	05/06/80	Fire protection system inoperable due to loss of power supply voltage
80-011/ 03L	04/07/80	05/07/80	Fire protection system voltage indication lost

REVIEWED BY OMPA LOEB: L. Tonson DATE: 7/3/80

REVIEWED BY NRR DOR: A. Burger DATE: 7/7/80

YANKEE ROWE

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-029
2. Utility Contact: F.C. BEERS (617) 366-9011
3. Licensed Thermal Power (MWt): 600
4. Nameplate Rating (Gross MWe): 185 x 1.0 = 185
5. Design Electrical Rating (Net MWe): 175
6. Maximum Dependable Capacity (Gross MWe): 180
7. Maximum Dependable Capacity (Net MWe): 175
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

CAPACITY FACTORS CALCULATED WITH A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	3,647.0	171,284.0
12. Hours Reactor Was Critical	.0	445.8	138,810.4
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	.0	439.7	134,467.1
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	253,769	72,002,091
17. Gross Electrical Energy (MWH)	0	80,635	22,152,788
18. Net Electrical Energy (MWH)	0	75,749	20,741,893
19. Unit Service Factor	.0	12.1	78.5
20. Unit Availability Factor	.0	12.1	78.5
21. Unit Capacity Factor (MDC Net)	.0	11.9	71.5
22. Unit Capacity Factor (DER Net)	.0	11.9	71.5
23. Unit Forced Outage Rate	100.0	86.8	3.6

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 09/15/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	License Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
1	01/19/80	F	744.0	A	4				THE PLANT IS STILL SHUTDOWN FOR TURBINE ROTOR INVESTIGATION AND REPAIR.

SUMMARY

THE PLANT REMAINED SHUTDOWN THE ENTIRE MONTH DUE TO TURBINE DISC FAILURE.

LEGEND

[1] Type:
F–Forced
S–Scheduled

[2] Reason:
A–Equipment Failure
B–Maintenance or Test
C–Refueling
D–Regulatory Restriction
E–Operator Training & License Examination
F–Administrative
G–Operational Error
H–Other

[3] Method:
1–Manual
2–Manual Scram
3–Automatic Scram
4–Continuations
5–Load Reduction
9–Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

ZION 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1. Location (state/county) (nearest population center)	Illinois/Lake 40 miles N of Chicago
2. Electric Reliability Council	Mid-America Interpool Network
3. Type of Reactor	PWR
4. Power Level (MWT)	3250
5. Design Electrical Rating (MWE Net)	1040
6. Date of Initial Criticality	June 19, 1973
7. Date of Commercial Operation	December 31, 1973
8. Condenser Cooling Method	Straight Through
9. Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10. Licensee	Commonwealth Edison Co.
11. Corporate Address	P. O. Box 767 Chicago, Illinois
12. Corporate Contact	D. L. Peoples, Manager of Nuclear Licensing
13. Architect/Engineer	Sargent & Lundy
14. Nuclear Steam Supply System	Westinghouse
15. Constructor	Various

Regulatory Information

16. IE Region responsible	III
17. IE Branch Chief	R. Heishman
18. IE Section Chief	W. S. LITTLE
19. IE Resident Inspector.	J. E. Kohler (312) 746-2313
20. Licensing Project Manager	E. Reeves
21. Docket Number	50-295
22. License Number/Date of Issue	DPR-39 issued 4/6/73
23. Public Document Room	Zion-Benton Public Library 2600 Emmans Avenue Zion, Illinois 60099

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

FACILITY ITEMS

Enclosed report contains licensee status on 60 day compliance items contained in February 29 1980 Confirmatory Order. (80-07)

MANAGERIAL ITEMS

Mr. C. Schuman, Operating Assistant Superintendent for Institute for Nuclear Power Plant Operations, Atlanta Georgia; Mr. L. Soth to Operating Assistant Superintendent; and Mr. G. Plime to Administrative Services and Technical Support Assistant Superintendent. (80-03)

PLANT STATUS

Unit 1 operating routinely.

LAST IE SITE INSPECTION DATE: April 14-30, 1980

INSPECTION REPORT NO: 80-07

SUMMARY

INSPECTION SUMMARY

Inspection on February 7 through March 28, (80-03): Routine resident inspection of plant operations, review of licensee events, plant modifications, compliance with confirmatory order, emergency procedure review, and non-licensed training. The inspection involved 122 inspector hours onsite by one NRC inspector. Of the areas inspected, one item of noncompliance was identified (violation of minimum shutdown margin requirements while in cold shutdown).

Inspection on March 18, (80-04): Nonroutine, announced inspection of confirmatory measurements program including: (1) a discussion of confirmatory measurement results from a previous sample collection, which included the submission of spiked samples, and (2) the collection of effluent samples and the submission of spiked samples for future comparative analyses. The inspection involved 8.5 inspector hours onsite by one NRC inspector. Within the areas inspected, no apparent items of noncompliance or deviations were noted.

Inspection on March 10-11 and April 17-18, (80-06): Small Break LOCA Procedure. Inspection involved 30 inspector hours onsite by two NRC inspectors. Of the areas inspected, no items of noncompliance were identified.

Inspection on April 14-30, (80-07): Announced inspection by the Resident Inspector of certain licensee actions taken to comply with the Confirmatory Order issued February 29, 1980. The inspection involved 20 inspector hours onsite by one NRC inspector. No items of noncompliance were identified.

REPORTS RECEIVED FROM LICENSEE

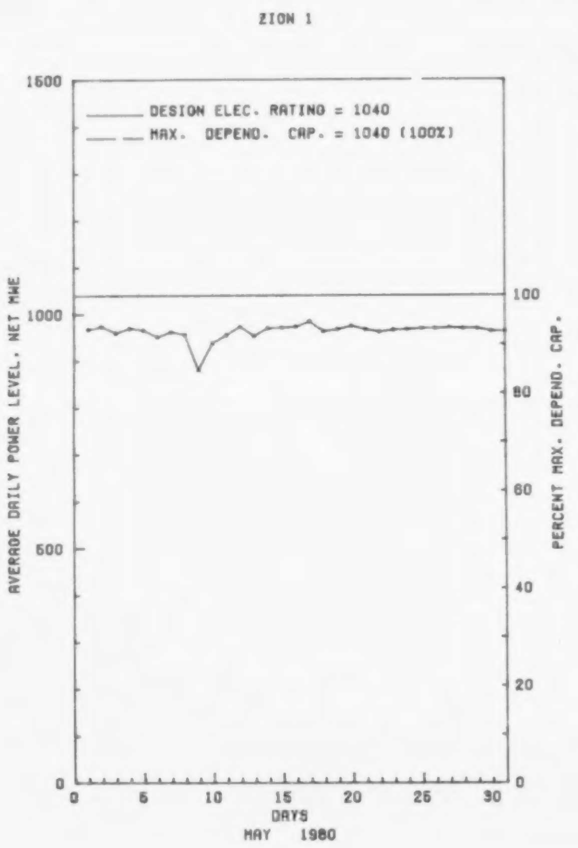
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-14/ 03L-0	03/20/80	04/18/80	During normal operation, blower of the containment vent rad monitor 1RE-0011, repeatedly tripped.
80-15/ 03L-0	04/01/80	04/22/80	Aux. bldg. exhaust system particulate iodine rad monitor ORT-PR12A&B instru. fail alarms activated.
80-16/ 03L-0	03/25/80	04/18/80	Unit NSO noted that S/G stm flow channel 1FT-523 was reading low greater than allowable channel tolerance (limit 4.5%).
80-17/ 03L-0	04/02/80	04/25/80	During loading test on 0 DG, it tripped from full load after running for 40 minutes.
80-18/ 03L-0	04/02/80	05/02/80	Both service water MOV's to AFW pump IC backup supply failed to stroke properly.
80-20/ 03L-0	04/03/80	05/02/80	Following a Unit 2 trip, the Unit 1 air ejector monitor 1RT PR26 failed.
80-21/ 03L-0	04/23/80	05/23/80	While operating at 98% power, monthly surveillance requiring perf. testing of the safeguards logic and hi pressure logic was performed 11 days late.

REVIEWED BY OMPA LOEB: L. Tomson **DATE:** 7/3/80

REVIEWED BY NRR DOR: Ed Reeves **DATE:** 7-7-80

ZION 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-295
2. Utility Contact: J. COOK (312) 746-2084
3. Licensed Thermal Power (MWt): 3250
4. Nameplate Rating (Gross MWe): 1220 x 0.9 = 1098
5. Design Electrical Rating (Net MWe): 1040
6. Maximum Dependable Capacity (Gross MWe): 1085
7. Maximum Dependable Capacity (Net MWe): 1040
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	3,647.0	56,255.0
12. Hours Reactor Was Critical	744.0	2,523.7	39,631.4
13. Reactor Reserve Shutdown Hours	.0	.0	2,621.8
14. Hours Generator On-Line	744.0	2,431.9	38,519.2
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	2,405,455	7,421,570	106,804,571
17. Gross Electrical Energy (MWH)	747,045	2,317,715	34,549,765
18. Net Electrical Energy (MWH)	716,174	2,193,096	32,677,342
19. Unit Service Factor	100.0	66.7	68.5
20. Unit Availability Factor	100.0	66.7	68.5
21. Unit Capacity Factor (MDC Net)	92.6	57.8	55.9
22. Unit Capacity Factor (DER Net)	92.6	57.8	55.9
23. Unit Forced Outage Rate	.0	33.3	14.7

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
9	05/09/80	F	.0	A	5		CH	PUMPXX	POWER REDUCED FOR FEED PUMP PROBLEMS.

SUMMARY

OPERATIONS WERE ROUTINE WITH A BRIEF POWER REDUCTION DUE TO FEED PUMP PROBLEMS.

LEGEND

[1]Type:
- F--Forced
- S--Scheduled

[2]Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3]Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuation
- 5--Load Reduction
- 9--Other

[4]Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5]Exhibit H: Same source.

ZION 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Illinois/Lake 40 miles N of Chicago
2.	Electric Reliability Council	Mid-America Interpool Network
3.	Type of Reactor	PWR
4.	Licensed Power Level (MWT)	3250
5.	Design Electrical Rating (MWE Net)	1040
6.	Date of Initial Criticality	December 24, 1973
7.	Date of Commercial Operation	September 17, 1974
8.	Condenser Cooling Method	Straight Through
9.	Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10.	Licensee	Commonwealth Edison Co.
11.	Corporate Address	P. O. Box 767 Chicago, Illinois
12.	Corporate Contact	D. L. Peoples, Manager of Nuclear Licensing
13.	Architect/Engineer	Sargent & Lundy
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Various

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	W. S. LITTLE
19.	IE Resident Inspector	J. E. Kohler (312) 746-2313
20.	Licensing Project Manager	E. Reeves
21.	Docket Number	50-304
22.	License Number/Date of Issue	DPR-48, 11/14/73
23.	Public Document Room	Zion-Benton Public Library 2600 Emmans Avenue Zion, Illinois 60099

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENTS

The "A" and "C" low pressure turbine spindles are being replaced during the refueling outage due to cracks which were found during inspection.

The steam generator seismic snubbers are being tested and modified during the outage.

FACILITY ITEMS

Enclosed report contains licensee status on 60 day compliance items contained in February 29, 1980 Confirmatory Order. (80-06)

MANAGERIAL ITEMS

Mr. C. Schuman, Operating Assistant Superintendent for Institute for Nuclear Power Plant Operations, Atlanta Georgia; Mr. L. Soth to Operating Assistant Superintendent; and Mr. G. Plime to Administrative Services and Technical Support Assistant Superintendent. (80-02)

PLANT STATUS

Unit began a routine refueling outage on May 2, 1980 scheduled to last for 56 days.

INSPECTION REPORT NO: 80-06

SUMMARY

INSPECTION SUMMARY

Inspection on February 7 through March 28, (80-02): Routine resident inspection of plant operations, review of licensee events, plant modifications, compliance with confirmatory order, emergency procedure review, and non-licensed training. The inspection involved 122 inspector hours onsite by one NRC inspector. Of the areas inspected, no items of noncompliance or deviations were identified.

Inspection on March 18, (80-03): Nonroutine, announced inspection of confirmatory measurements program including: (1) a discussion of confirmatory measurement results from a previous sample collection, which included the submission of spiked samples, and (2) the collection of effluent samples and the submission of spiked samples for future comparative analyses. The inspection involved 8.5 inspector hours onsite by one NRC inspector. Within the areas inspected, no apparent items of noncompliance or deviations were noted.

Inspection on March 10-11 and April 17-18, (80-05): Small Break LOCA Procedure. Inspection involved 30 inspector hours onsite by two NRC inspectors. Of the areas inspected, no items of noncompliance were identified.

Inspection on April 14-30, (80-06): Announced inspection by the Resident Inspector of certain licensee actions taken to comply with the Confirmatory Order issued February 29, 1980. The inspection involved 20 inspector hours onsite by one NRC inspector. No items of noncompliance were identified.

REPORTS RECEIVED FROM LICENSEE

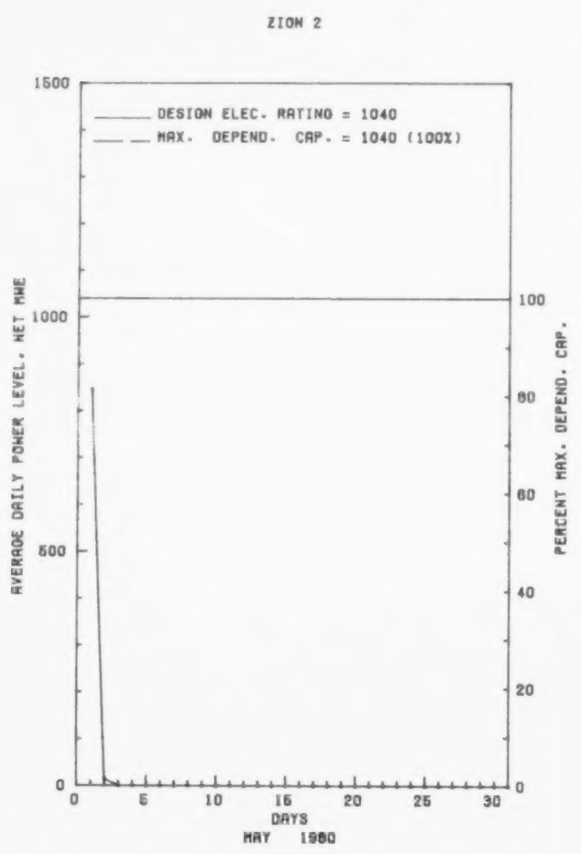
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-17/ 03L-0	04/10/80	05/09/80	During Grd. checking on DC Bus 212CK12 (Normal feed DC control pwr to bus 249) was opened and would not reclose.

REVIEWED BY OMPA LOEB: L. Tomeon DATE: 7/3/80

REVIEWED BY NRR DOR: Ed Reeves DATE: 7-7-80

ZION 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-304
2. Utility Contact: J. COOK (312) 746-2084
3. Licensed Thermal Power (MWt): 3250
4. Nameplate Rating (Gross MWe): 1220 x 0.9 = 1098
5. Design Electrical Rating (Net MWe): 1040
6. Maximum Dependable Capacity (Gross MWe): 1085
7. Maximum Dependable Capacity (Net MWe): 1040
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	3,647.0	49,968.0
12. Hours Reactor Was Critical	28.2	2,439.0	36,042.6
13. Reactor Reserve Shutdown Hours	.0	.0	226.1
14. Hours Generator On-Line	27.8	2,409.1	35,223.0
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	67,827	7,382,048	100,017,366
17. Gross Electrical Energy (MWH)	22,240	2,417,965	32,139,185
18. Net Electrical Energy (MWH)	16,266	2,296,176	30,497,634
19. Unit Service Factor	3.7	66.1	70.5
20. Unit Availability Factor	3.7	66.1	70.5
21. Unit Capacity Factor (MDC Net)	2.1	60.5	58.7
22. Unit Capacity Factor (DER Net)	2.1	60.5	58.7
23. Unit Forced Outage Rate	.0	17.8	17.7

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUELING STARTED MAY 2, 1980, FOR APPROX. 8 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: 06/27/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
12	05/02/80	S	716.2	C	1		RC	FUELXX	START OF REFUELING OUTAGE.

SUMMARY

UNIT SHUTDOWN ON 05/02/80 FOR ANNUAL REFUELING.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

SECTION 3 – APPENDIX

STATUS OF SPENT FUEL STORAGE CAPABILITY

DATA AS OF 5-31-80

Facility	Core Size[a] (No. of Assemblies)	Present Authorized Storage Pool Capacity (Fuel Assemblies)	No. of Assemblies Stored	Remaining Capacity (No. of Assemblies)	Remaining Capacity If Pending Request Approved (No. of Assemblies)	Scheduled Date For Next Refueling	Will Fill Present[b] Authorized Capacity
I. Pressurized Water Reactors							
ARKANSAS 1	177	590	176	414		01-81	1988
ARKANSAS 2	177	486	58	428		03-81	1989
BEAVER VALLEY 1	157	833	0	833		N/S	1995
CALVERT CLIFFS 1	217	1056 (c)	364 (c)	692 (c)		10-80	1984
CALVERT CLIFFS 2	217					01-81	
COOK 1	193	500	193	307	1857	N/S	1985
COOK 2	193					N/S	
CRYSTAL RIVER 3	177	256	112	144	1109	N/S	1982
DAVIS-BESSE 1	177	735	52	683		N/S	1989
FARLEY 1	157	675	0	675		09-80	1990
FORT CALHOUN 1	133	483	197	286		03-81	1985
GINNA	121	595	156	439		N/S	1987
HADDAM NECK	157	1168	389	779		09-81	1994
INDIAN POINT 1	0	288	160	128		N/S	
INDIAN POINT 2	193	482	200	282		N/S	1984
INDIAN POINT 3	193	837	64	773		N/S	1989
KEWAUNEE	121	990	120	870		N/S	1999
MAINE YANKEE	217	953	433	520		N/S	1986
MILLSTONE 2	217	667	144	523		07-80	1987
NORTH ANNA 1	157	400	0	400	966	N/S	1984
NORTH ANNA 2 (d)	157	400	0	400		N/S	
OCONEE 1	177	1224 (l)	686	538 (l)	755	02-81	1980
OCONEE 2	177					N/S	
OCONEE 3	177					12-80	
PALISADES	204	798	273	525		N/S	1985
POINT BEACH 1	121	1502 (c)	276 (c)	1226 (c)		11-80	1995
POINT BEACH 2	121					03-81	
PRAIRIE ISLAND 1	121	687 (c)	280 (c)	407 (c)		08-80	1984
PRAIRIE ISLAND 2	121					02-81	
RANCHO SECO 1	177	579	164	415		07-81	1987
ROBINSON 2	157	272	106	166 (e)		07-80	1980 (g)
SALEM 1	193	264	40	224	1130	09-80	1982
SAN ONOFRE 1	157	216	97	119		N/S	1983
ST LUCIE 1	217	728	60	668		N/S	1990
SURRY 1	157	1044 (c)	1983 (c)	608 (c)		09-80	1983
SURRY 2	157					09-80	
THREE MILE ISLAND 1	177	752	208	544		N/S	1989
THREE MILE ISLAND 2	177	442	0	442		N/S	1986
TROJAN	193	651	64	587		N/S	1988
TURKEY POINT 3	157	621	346 (c)	275 (c)		N/S	1987
TURKEY POINT 4	157					N/S	
YANKEE-ROWE 1	76	391	149	242	572	N/S	1985
ZION 1	193	868 (c)	508 (c)	1604 (c)	1676	01-81	1993
ZION 2	193					N/S	

Facility	Core Size[a] (No. of Assemblies)	Present Authorized Storage Pool Capacity (Fuel Assemblies)	No. of Assemblies Stored	Remaining Capacity (No. of Assemblies)	Remaining Capacity If Pending Request Approved (No. of Assemblies)	Scheduled Date For Next Refueling	~~Will Fill Present~~[k] ~~Authorized Capacity~~
II. Boiling Water Reactors							
III. Independent Spent Fuel Storage Installations (h)							
Morris Operations		750 MTU (j)	315	385 MTU (j)	1490 MTU (j)		
NFS(i)		250 MTU	170 MTU	80 MTU			

(a) At each refueling outage approximately 1/3 of a PWR core and 1/4 of a BWR core is off-loaded.
(b) Some of these dates have been adjusted by staff assumptions.
(c) This is the total for both units.
(d) Plant not in commercial operation.
(e) Some spent fuel stored at Brunswick.
(f) Authorized a total 2088 BWR and 304 PWR assemblies for both pools.
(g) Robinson 2 assemblies being shipped to Brunswick for storage.
(h) Capacity is in metric tons of uranium; 1 MTU = 2 PWR assemblies or 5 BWR assemblies.
(i) No longer accepting spent fuel.
(j) Racked for 700 MTU.
(k) ~~Reserved~~
(l) This is the station total.

N/S = Not Scheduled

REACTOR-YEARS OF EXPERIENCE

AS OF 5-31-80

The total of Reactor years of experience is as the sum of all calendar days for each unit, from the date that electricity was first generated until a final shutdown date or the status date, whichever comes first, divided by 365.25 days/year.

If a date is unknown, the first day of the first month of operation is substituted. Units which have not yet generated electricity but which are licensed are listed but not included in the computation.

LICENSED OPERATING ELECTRICAL PRODUCING UNITS

UNIT NAME	FIRST ELEC. GENERATION	YEARS EXPERIENCE
ARKANSAS 1	08/01/74	5.83
ARKANSAS 2	12/26/78	1.18
BEAVER VALLEY 1	06/14/76	3.96
BIG ROCK POINT 1	12/08/62	17.48
BROWNS FERRY 1	10/15/73	6.63
BROWNS FERRY 2	08/28/74	5.76
BROWNS FERRY 3	09/12/76	3.72
BRUNSWICK 1	12/04/76	3.49
BRUNSWICK 2	04/29/75	5.09
CALVERT CLIFFS 1	01/03/75	5.41
CALVERT CLIFFS 2	12/07/76	3.48
COOK 1	02/10/75	5.31
COOK 2	03/22/78	2.20
COOPER STATION	05/10/74	6.06
CRYSTAL RIVER 3	01/30/77	3.33
DAVIS-BESSE 1	08/28/77	2.76
DRESDEN 1	04/15/60	20.13
DRESDEN 2	04/13/70	10.14
DRESDEN 3	07/22/71	8.86
DUANE ARNOLD	05/19/74	6.04
FARLEY 1	08/18/77	2.79
FITZPATRICK	02/01/75	5.33
FORT CALHOUN 1	08/25/73	6.77
FORT ST VRAIN	12/11/76	3.47
GINNA	12/02/69	10.50
HADDAM NECK	08/07/67	12.82
HATCH 1	11/11/74	5.56
HATCH 2	09/22/78	1.69
INDIAN POINT 2	06/26/73	6.93
INDIAN POINT 3	04/27/76	4.10
KEWAUNEE	04/08/74	6.15
LA CROSSE	04/26/68	12.10
MAINE YANKEE	11/08/72	7.56
MILLSTONE 1	11/29/70	9.51
MILLSTONE 2	11/09/75	4.56
MONTICELLO	03/05/71	9.24
NINE MILE POINT 1	11/09/69	10.56
NORTH ANNA 1	04/17/78	2.12
OCONEE 1	05/06/73	7.07
OCONEE 2	12/05/73	6.49
OCONEE 3	09/01/74	5.75
OYSTER CREEK 1	09/23/69	10.69
PALISADES	12/31/71	8.42
PEACH BOTTOM 2	02/18/74	6.28
PEACH BOTTOM 3	09/01/74	5.75
PILGRIM 1	07/19/72	7.87
POINT BEACH 1	11/06/70	9.57
POINT BEACH 2	08/02/72	7.83
PRAIRIE ISLAND 1	12/04/73	6.49
PRAIRIE ISLAND 2	12/21/74	5.45
QUAD CITIES 1	04/12/72	8.14
QUAD CITIES 2	05/23/72	8.02
RANCHO SECO 1	10/13/74	5.63
ROBINSON 2	09/26/70	9.68
SALEM 1	12/25/76	3.43

LICENSED OPERATING ELECTRICAL PRODUCING UNITS - CONT.

UNIT NAME	FIRST ELEC. GENERATION	YEARS EXPERIENCE
SAN ONOFRE 1	07/16/67	12.88
ST LUCIE 1	05/07/76	4.07
SURRY 1	07/04/72	7.91
SURRY 2	03/10/73	7.23
THREE MILE ISLAND 1	06/19/74	5.95
TROJAN	12/23/75	4.44
TURKEY POINT 3	11/02/72	7.58
TURKEY POINT 4	06/21/73	6.95
VERMONT YANKEE 1	09/20/72	7.70
YANKEE-ROWE 1	11/10/60	19.56
ZION 1	06/28/73	6.93
ZION 2	12/26/73	6.43
TOTAL		468.79 YEARS

PERMANENTLY OR INDEFINITELY SHUTDOWN UNITS

UNIT NAME	FIRST ELEC. GENERATION	SHUTDOWN OR ELECTRICAL CESSATION DATE	YEARS EXPERIENCE
BONUS	08/14/64	06/01/68	3.80
CVTR	12/18/63	01/01/67	3.04
ELK RIVER	08/24/63	02/01/68	4.44
FERMI 1	08/05/66	11/29/72	6.32
HALLAM	05/29/63	09/01/64	1.26
HUMBOLDT BAY	04/18/63	07/02/76	13.21
INDIAN POINT 1	09/16/62	10/31/74	12.12
PATHFINDER	07/25/66	10/01/67	1.19
PEACH BOTTOM 1	01/27/67	11/01/74	7.76
PIQUA	11/04/63	01/01/66	2.16
THREE MILE ISLAND 2	04/21/78	03/28/79	.93
TOTAL			56.23 YEARS

NON–POWER REACTORS IN THE U.S.

1. RESEARCH REACTORS

State	Location	Licensee	Reactor Type	Docket	License Number	Date Issued	Authorized Power Level (KW)
Alabama	1. Tuskegee	Tuskegee Institute	AGN 201 No. 102	50-406	R-122	08-30-74	.0001
Arizona	2. Tucson	U. of Arizona	TRIGA Mk. I	50-113	R-52	12-05-58	100
California	3. Los Angeles	U. of California	Argonaut	50-142	R-71	10-03-60	100
	4. Berkeley	U. of California	TRIGA Mark III	50-224	R-101	08-10-66	1000
	5. Irvine	U. of California	TRIGA Mark I	50-326	R-116	11-24-69	250
	6. Santa Barbara	U. of California	L-77	50-433	R-124	12-03-74	.01
	7. Canoga Park	Rockwell International Corporation	L-77	50-94	R-40	05-17-58	.01
	8. Hawthorne	Northrop Corporate Laboratories	TRIGA Mark F	50-187	R-90	03-04-63	1000
	9. San Luis Obispo	California State Polytechnic College	AGN 201 No. 100	50-394	R-121	05-16-73	.0001
	10. Alameda County	General Electric Co.	NTR	50-73	R-33	10-31-57	100
	11. San Diego	General Atomic Co.	TRIGA Mark I	50-89	R-38	05-03-58	250
	12. San Diego	General Atomic Co.	TRIGA Mark F	50-163	R-67	07-01-60	1500
	13. San Ramon	Aerotest Operations, Inc.	TRIGA (Industrial)	50-228	R-98	07-02-65	250
Colorado	14. Jefferson County	U.S. Geological Survey. Dept. of Interior	TRIGA Mark I	50-274	R-113	02-24-69	1000
Delaware	15. Newark	U. of Delaware	AGN-201 No. 113	50-98	R-43	07-03-58	.0001
District of Columbia	16. –	Catholic U.	AGN-201 No. 101	50-77	R-31	11-15-67	.0001
Florida	17. Gainesville	U. of Florida	Argonaut	50-83	R-56	05-21-59	100
Georgia	18. Atlanta	Georgia Institute of Technology	AGN-201 No. 104	50-276	R-111	04-19-68	.0001
	19. Atlanta	Georgia Institute of Technology	Heavy Water	50-160	R-97	12-29-64	5000
Idaho	20. Pocatello	Idaho State U.	AGN-201 No. 103	50-284	R-110	10-11-67	.0001
Illinois	21. Urbana	U. of Illinois	Advanced TRIGA	50-151	R-115	07-22-69	1500
	22. Urbana	U. of Illinois	LOPRA	50-356	R-117	12-27-71	10
	23. Zion	Westinghouse Electric Corporation	NTR	50-87	R-119	01-28-72	10
Indiana	24. West Lafayette	Purdue U.	Lockheed	50-182	R-87	08-16-62	10

RESEARCH REACTORS – CONT.

State	Location	Licensee	Reactor Type	Docket	License Number	Date Issued	Authorized Power Level (KW)
Iowa	25. Ames	Iowa State U.	UTR-10	50-116	R-59	10-16-59	10
Kansas	26. Lawrence	U. of Kansas	Lockheed	50-148	R-78	06-23-61	250
	27. Manhattan	Kansas State U.	TRIGA	50-188	R-88	10-16-62	250
Maryland	28. College Park	U. of Maryland	TRIGA	50-166	R-70	10-14-60	250
	29. Bethesda	Armed Forces Radiobiology Research Institute	TRIGA	50-170	R-84	06-26-62	1000
Massachusetts	30. Lowell	U. of Lowell	GE	50-223	R-125	12-24-74	1000
	31. Cambridge	Massachusetts Institute of Technology	Heavy Water Reflected	50-20	R-37	06-09-58	5000
	32. Worcester	Worcester Polytechnic Institute	GE	50-134	R-61	12-16-59	10
Michigan	33. Ann Arbor	U. of Michigan	Pool	50-2	R-28	09-13-57	2000
	34. Lansing	Michigan State U.	TRIGA Mark I	50-294	R-114	03-21-69	250
	35. Midland	Dow Chemical Co.	TRIGA Mark I	50-264	R-108	07-03-67	100
Missouri	36. Rolla	U. of Missouri	Pool	50-123	R-79	11-21-61	200
	37. Columbia	U. of Missouri	Tank	50-186	R-103	10-11-66	10,000
Nebraska	38. Omaha	V.A. Hospital	TRIGA	50-131	R-57	06-26-59	18
New Mexico	39. Albuquerque	U. of New Mexico	AGN-201M No. 112	50-252	R-102	09-17-66	.005
New York	40. Buffalo	State U. of New York	PULSTAR	50-57	R-77	03-24-61	2000
	41. Sterling Forest	Union Carbide Corp.	Pool	50-54	R-81	09-07-61	5000
	42. New York City	Columbia U.	TRIGA Mark II	50-208	R-128	04-14-77	250
	43. Ithaca	Cornell U.	ZPR	50-97	R-89	12-11-62	.1
	44. Ithaca	Cornell U.	TRIGA Mk. II	50-157	R-80	01-11-62	100
	45. Riverdale	Manhattan College	Tank	50-199	R-94	03-24-64	.0001
North Carolina	46. Raleigh	North Carolina State U.	PULSTAR	50-297	R-120	08-25-72	1000
Ohio	47. Columbus	Ohio State U.	Pool	50-150	R-75	02-24-61	10
Oklahoma	48. Norman	U. of Oklahoma	AGN-211 No. 102	50-112	R-53	12-29-58	.015
Oregon	49. Corvallis	Oregon State U.	AGN-201 No. 114	50-106	R-51	11-13-58	.0001
	50. Corvallis	Oregon State U.	TRIGA Mark II	50-243	R-106	03-07-67	1000
	51. Portland	Reed Institute	TRIGA Mk. I	50-288	R-112	07-02-68	250
Pennsylvania	52. University Park	Pennsylvania State U.	TRIGA Mk. III	50-5	R-2	07-08-55	1000

NON–POWER REACTORS IN THE U.S.

RESEARCH REACTORS – CONT.

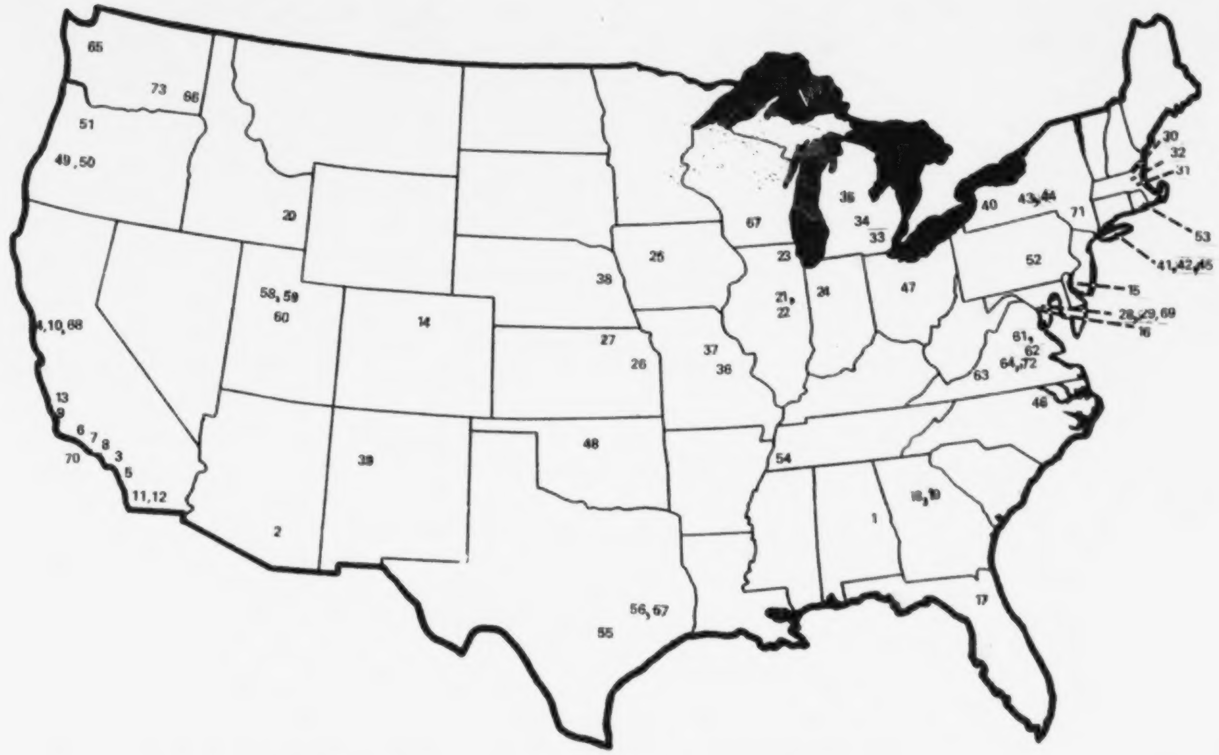
State	Location	Licensee	Reactor Type	Docket	License Number	Date Issued	Authorized Power Level (KW)
Rhode Island	53. Ft. Kearney, Narragansett	Rhode Island & Providence Plantations AEC	GE Pool	50-193	R-95	07-21-64	2000
Tennessee	54. Shelby County	Memphis State U.	AGN-201 No. 108	50-538	R-127	12-10-76	.0001
Texas	55. Austin	U. of Texas	TRIGA Mk. I	50-192	R-92	08-02-63	250
	56. College Station	Texas A&M U.	TRIGA	50-128	R-83	12-07-61	1000
	57. College Station	Texas A&M U.	AGN- 201M No. 106	50-59	R-23	08-26-57	.005
Utah	58. Salt Lake City	U. of Utah	AGN-201M No. 107	50-72	R-25	09-12-57	.005
	59. Salt Lake City	U. of Utah	TRIGA Mark I	50-407	R-126	09-30-75	100
	60. Provo	Brigham Young U.	L-77	50-262	R-109	09-07-67	.01
Virgina	61. Charlottesville	U. of Virgina	Pool	50-62	R-66	06-27-60	2000
	62. Charlottesville	U. of Virginia	Cavalier	50-396	R-123	09-24-74	.1
	63. Blacksburg	Virginia Polytechnic Institute	UTR-10	50-124	R-62	12-18-59	100
	64. Lynchburg	Babcock & Wilcox Co.	LPR	50-99	R-47	09-05-58	1000
Washington	65. Seattle	U. of Washington	Argonaut	50-139	R-73	03-31-61	100
	66. Pullman	Washington State U.	TRIGA	50-27	R-76	03-06-61	1000
Wisconsin	67. Madison	U. of Wisconsin	TRIGA	50-156	R-74	11-23-60	1000

2. EXPERIMENTAL AND TEST REACTORS

State	Location	Licensee	Reactor Type	Docket	License Number	Date Issued	Authorized Power Level (MWt)
California	68. Alameda Co.	General Electric Co.	GETR	50-70	TR-1	01-07-59	50
Maryland	69. Gaithersburg	National Bureau of Standards	TEST	50-184	TR-5	11-13-67 POL 06-30-70 OL	10

3. CRITICAL EXPERIMENT FACILITIES

State	Location	Licensee	Docket	License Number	Date Issued	Experiments
California	70. Ventura Co.	Rockwell International Corp. (Possesion Only)	50-147	CX-17	10-03-60	AETR Experiments (Terminal Requested).
New York	71. Schenectady	Rensselaer Polytechnic Institute	50-225	CX-22	07-03-64	
Virginia	72. Lynchburg	Babcock & Wilcox Co.	50-13	CX-10	10-22-58	NMSR Experiments
Washington	73. Richland	Battelle Memorial Institute	50-360	CX-26	11-29-71	Plutonium Recycle Critical Facility



Source: NRC Facilities License Application Record (FLAR) 06-30-77.

ABBREVIATIONS

AGN	Aerojet General Nuclear	These reactors were constructed by Aerojet at San Ramon, California, and then transferred to the current location.
ZPR	Zero Power Reactor	
LPR	Low Power Reactor	

DISTRIBUTION LIST–NRC

INTERNAL DISTRIBUTION

Office of the Commissioners	5
Atomic Safety and Licensing Appeal Panel	3
Advisory Committee on Reactor Safeguards	6
Office of Inspector and Auditor	1
Office of Policy Evaluation	1
Office of the General Counsel	1
Office of Public Affairs	21
Office of Congressional Affairs	3
Office of the Executive Director for Operations	7
Office of Administration	2
Office of the Executive Legal Director	2
Office of the Controller	2
Office for Analysis and Evaluation of Operational Data	2
Office of International Programs	3
Office of State Programs	1
Office of Management and Program Analysis	14
Office of Standards Development	1
Office of Nuclear Material Safety and Safeguards	2
– Division of Fuel Cycle and Material Safety	3
– Division of Safeguards	2
Office of Nuclear Reactor Regulation	5
– Division of Project Management	6
– Division of Operating Reactors	105
– Division of Systems Safety	4
– Division of Site Safety and Environmental Analysis	18
Office of Nuclear Regulatory Research	4
Office of Inspection and Enforcement	7
– Region I	11
– Region II	10
– Region III	10
– Region IV	10
– Region V	7
	279

EXTERNAL DISTRIBUTION

SPECIAL REQUESTS

Air Force	1
Congress	10
Department of Energy	32
Government Accounting Office	1
Environmental Protection Agency	1
Electric Power Research Institute	1
Argonne National Laboratory	3
Federal Power Commission	1
Bureau of Mines	1
Department of Agriculture	1
Department of Commerce	1
Department of the Interior	1
	54

OTHER

GPO Depository	440
GPO Stores	75
National Technical Information Service	25
Subscriptions (NTIS)	250
Colleges and Libraries (including Public Document Rooms)	128
Utilities and Other Requests	227
	1145

best copy available

15/4-7

NUREG 0020
VOL. 4 NO. 6
JULY 1980

Nuclear Regulatory Commission

OPERATING UNITS STATUS REPORT

DATA AS OF 6-30-80

NEWARK PUBLIC LIBRARY
SEP 23 1980
U. S. GOVT. PUBS.
DEPOSITORY 376

LICENSED OPERATING REACTORS

Y3.N88:

Available from
GPO Sales Program
Division of Technical Information and Document Control
U. S. Nuclear Regulatory Commission
Washington, D.C. 20555
and
National Technical Information Service
Springfield, Virginia 22161
703-326-8543
A Year's Subscription, 12 Issues, for this Publication is
$120.00 North American Continent ($240 00 Other).
Multiple Subscriptions Entered at the Same Time to the Same Address
Cost $90.00 North American Continent ($180.00 Other) for each Additional Subscription.

Single copies are $12.50 North American Continent ($25.00 Other).

This document is provided free of charge to those who participate in the data collection program, and to the Nuclear Regulatory Commission's local public document rooms.

Issued by the U S. Nuclear Regulatory Commission

PUBLISHED AUGUST 8,1980

Nuclear Regulatory Commission

OPERATING UNITS STATUS REPORT

DATA AS OF 6-30-80

NUREG 0020
VOL. 4 NO. 6
JULY 1980

LICENSED OPERATING REACTORS

READEX FICHE

AUTHORIZATION AND CLEARANCE

THE U.S. NUCLEAR REGULATORY COMMISSION'S OFFICE OF MANAGEMENT AND PROGRAM ANALYSIS PUBLISHES THIS MONTHLY STATUS REPORT "AS PART OF THE REPORTING REQUIREMENTS IN SECTION 50.36 OF 10 CFR PART 50 UNDER GAO CLEARANCE NUMBER B-180225, WITH AN EXPIRATION DATE OF SEPTEMBER 30, 1981," AS STATED IN THE OCTOBER 3, 1978 LETTER FROM JOHN M. LOVELADY, ASSISTANT DIRECTOR, GENERAL GOVERNMENT DIVISION, U. S. GENERAL ACCOUNTING OFFICE, TO J.M. FELTON, DIRECTOR, DIVISION OF RULES AND RECORDS, U.S. NUCLEAR REGULATORY COMMISSION.

STATEMENT OF PURPOSE

The U. S. Nuclear Regulatory Commission's monthly OPERATING UNITS STATUS REPORT – LICENSED OPERATING REACTORS, provides data on the operation of nuclear units as timely and accurately as possible. This information is collected by the Office of Management and Program Analysis from the Headquarters Staff of NRC's Office of Inspection and Enforcement, from NRC's Regional Offices, and from utilities. Since all of the data concerning operation of the units is provided by the utility operators less than two weeks after the end of the month, necessary corrections to published information are shown on the ERRATA page.

This report is divided into three sections: the first contains monthly highlights and statistics for commercial operating units, and errata from previously reported data; the second is a compilation of detailed information on each unit, provided by NRC Regional Offices, IE Headquarters and the Utilities; and the third section is an appendix for miscellaneous information such as spent fuel storage capability, reactor-years of experience and non-power reactors in the United States.

The percentage computations, Items 19 through 24 in Section 2, the vendor capacity factors on page 1-4, and actual vs. potential energy production on Page 1-2 are computed from actual power production from the period of consideration. The percentages listed in power generation on Page 1-2 are computed as an arithmetical average. The factors for the life-span of each unit (the "Cumulative" column) are reported by the utility and are not entirely re-computed by NRC. Utility power production data is checked for consistency with previously submitted statistics.

In some cases, the numbers in the "Cumulative" column may have been totaled from a date that is different from the date of commercial operation. In these cases, the column is noted with an explanation of the nature of the change on each individual plant page in Section 2.

It is hoped this status report proves informative and helpful to all agencies and individuals interested in analyzing trends in the nuclear industry which might have safety implications, or in maintaining an awareness of the U. S. energy situation as a whole.

TABLE OF CONTENTS

GLOSSARY ii
INDEX TO LICENSED UNITS iii

SECTION 1 – CURRENT DATA SUMMARIES

MONTHLY HIGHLIGHTS OF COMMERCIAL NUCLEAR POWER UNITS 1-2

- Licensed Power Reactors 1-2
- Power Generation 1-2
- Actual vs. Potential Energy Production 1-2
- Outage Data 1-2
- Derated Units 1-2
- Shutdowns - Number and Reasons 1-3
- Unit Availability, Capacity, and Forced Outage Rate 1-3
- Refueling Schedule 1-3

AVERAGE CAPACITY FACTORS BY VENDOR 1-4

AVERAGE DAILY POWER LEVEL FOR ALL COMMERCIAL OPERATING UNITS 1-5

ERRATA 1-7

SECTION 2 – OPERATING POWER REACTORS

ARKANSAS 1 THROUGH ZION 2 PAGES A–2 TO Z–5

- Facility Data
- Inspection and Enforcement Status
- Reports Received from Licensee
- Operating Status
- Average Daily Power Level (MWe)
- Unit Shutdowns/Reductions

SECTION 3 – APPENDIX

STATUS OF SPENT FUEL STORAGE CAPABILITY 3-2
REACTOR-YEARS OF EXPERIENCE 3-3
NON-POWER REACTORS IN THE U.S. 3-4

GLOSSARY

AVERAGE DAILY POWER LEVEL (MWe)	The net electrical energy generated during the day (measured from 0001 to 2400 hours inclusive) in megawatts hours divided by 24 hours.
LICENSED THERMAL POWER (MWt)	The maximum thermal power of the reactor authorized by the NRC, expressed in megawatts.
DATE OF COMMERCIAL OPERATION	Date unit was declared by utility owner to be available for the regular production of electricity; usually related to satisfactory completion of qualification tests as specified in the purchase contract and to accounting policies and practices of utility.
DESIGN ELECTRICAL RATING (DER) (Net MWe)	The nominal net electrical output of the unit specified by the utility and used for the purpose of plant design.
FORCED OUTAGE	An outage required to be initiated no later than the weekend following discovery of an off-normal condition.
FORCED OUTAGE HOURS	The clock hours during the report period that a unit is unavailable due to forced outages.
GROSS ELECTRICAL ENERGY GENERATED (MWH)	Electrical output of the unit during the report period as measured at the output terminals of the turbine generator, in megawatts hours.
GROSS HOURS	The clock hours from the beginning of a specified situation until its end. For outage durations, the clock hours during which the unit is not in power production.
GROSS THERMAL ENERGY GENERATED (MWH)	The thermal energy produced by the unit during the report period as measured or computed by the licensee in megawatt hours.
HOURS GENERATOR ON-LINE	Also, "Unit Service Hours." The total clock hours in the report period during which the unit operated with breakers closed to the station bus. These hours added to the total outage hours experienced by the unit during the report period, shall equal the hours in the report period.
HOURS IN REPORTING PERIOD	For units in power ascension at the end of the period, the gross hours from the beginning of the period or the first electrical production, whichever comes last, to the end of the period. For units in commercial operation at the end of the period, the gross hours from the beginning of the period or of commercial operation, whichever comes last, to the end of the period or decommissioning, whichever comes first.
HOURS REACTOR CRITICAL	The total clock hours in the report period during which the reactor sustained a controlled chain reaction.
MAXIMUM DEPENDABLE CAPACITY (GROSS) (MDC Gross) (Gross MWe)	Dependable main-unit gross capacity, winter or summer, whichever is smaller. The dependable capacity varies because the unit efficiency varies during the year due to cooling water temperature variations. It is the gross electrical output as measured at the output terminals of the turbine generator during the most restrictive seasonal conditions (usually summer).
MAXIMUM DEPENDABLE CAPACITY (NET) (MDC Net) (Net MWe)	Maximum Dependable Capacity (Gross) less the normal station service loads.
NAMEPLATE RATING (Gross MWe)	The nameplate power designation of the generator in megavolt amperes (MVA) times the nameplate power factor of the generator. NOTE: The nameplate rating of the generator may not be indicative of the maximum or dependable capacity, since some other item of equipment of a lesser rating (e.g., turbine) may limit unit output.
NET ELECTRICAL ENERGY GENERATED	Gross electrical output of the unit measured at the output terminals of the turbine generator during the reporting period, minus the normal station service electrical energy utilization. If this quantity is less than zero, a negative number should be recorded.
OUTAGE	A situation in which no electrical production takes place.
OUTAGE DATE	As reported on Appendix D of Reg. Guide 1.16, the date of the start of the outage. If continued from a previous month, report the same outage date but change "Method of Shutting Down Reactor" to "4 (Other)" and add a note: "Continued from previous month."
OUTAGE DURATION	The total clock hours of the outage measured from the beginning of the report period or the outage, whichever comes last, to the end of the report period or the outage, whichever comes first.
OUTAGE NUMBER	A number unique to the outage assigned by the licensee. The same number is reported each month in which the outage is in progress. One format is "76-05" for the fifth outage to occur in 1976.
PERIOD HOURS	See "Hours in Reporting Period."
POWER REDUCTION	A reduction in the Average Daily Power Level of more than 20% from the previous day. All power reductions are defined as outage of zero hours durations for the purpose of computing unit service and availability factors, and forced outage rate.

REACTOR AVAILABLE HOURS	The total clock hours in the report period during which the reactor was critical or was capable of being made critical. (Reactor Reserve Shutdown Hours + Hours Reactor Critical.)
REACTOR AVAILABILITY FACTOR	$\frac{\text{Reactor Available Hours} \times 100}{\text{Period Hours}}$
REACTOR RESERVE SHUTDOWN	The cessation of criticality in the reactor for administrative or other similar reasons when operation could have been continued.
REACTOR RESERVE SHUTDOWN HOURS	The total clock hours in the report period during which the reactor is in reserve shutdown mode. NOTE: No credit is given for NRC imposed shutdowns.
REACTOR SERVICE FACTOR	$\frac{\text{Hours Reactor Critical} \times 100}{\text{Period Hours}}$
REPORT PERIOD	Usually, the preceding calendar month. Can also be the preceding calendar year, (Year-to-Date), or the life-span of a unit (cumulative).
RESTRICTED POWER LEVEL	Maximum net electrical generation to which the unit is restricted during the report period due to the state of equipment, external conditions, administrative reasons, or a direction by NRC.
SCHEDULED OUTAGE	Planned removal of a unit from service for refueling, inspection, training, or maintenance. Those outages which do not fit the definition of "Forced Outage" perforce are "Scheduled Outages."
STARTUP AND POWER ASCENSION TEST PHASE	Period following initial criticality during which the unit is tested at successively higher levels, culminating with operation at full power for a sustained period and completion of warranty runs. Following this phase, the utility generally considers the unit to be available for commercial operation.
UNIT	The set of equipment uniquely associated with the reactor, including turbine generators, and ancillary equipment, considered as a single electrical energy production facility.
UNIT AVAILABLE HOURS	The total clock hours in the report period during which the unit operated on-line or was capable of such operation. (Unit Reserve Shutdown Hours + Hours Generator On-Line.)
UNIT AVAILABILITY FACTOR*	$\frac{\text{Unit Available Hours} \times 100}{\text{Period Hours}}$
UNIT CAPACITY FACTORS	
– Using Licensed Thermal Power	$\frac{\text{Gross Thermal Energy Generated} \times 100}{\text{Period Hours} \times \text{Lic. Thermal Power}}$
– Using Nameplate Rating	$\frac{\text{Gross Electrical Energy Generated} \times 100}{\text{Period Hours} \times \text{Nameplate Rating}}$
– Using DER	$\frac{\text{Net Electrical Energy Generated} \times 100}{\text{Period Hours} \times \text{DER}}$
– Using MDC Gross	$\frac{\text{Gross Electrical Energy Generated} \times 100}{\text{Period Hours} \times \text{MDC Gross}}$
– Using MDC Net	$\frac{\text{Net Electrical Energy Generated} \times 100}{\text{Period Hours} \times \text{MDC Net}}$

NOTE: if MDC Gross and/or MDC Net have not been determined, the DER is substituted for this quantity for Unit Capacity Factor calculations.

UNIT FORCED OUTAGE RATE	$\frac{\text{Forced Outage Hours}}{\text{Unit Service Hours} + \text{Forced Outage Hours}}$
UNIT RESERVE SHUTDOWN	The removal of the unit from on-line operation for economic or other similar reasons when operation could have been continued.
UNIT RESERVE SHUTDOWN HOURS	The total clock hours in the report period during which the unit was in reserve shutdown mode.
UNIT SERVICE FACTOR	$\frac{\text{Unit Service Hours} \times 100}{\text{Period Hours}}$
UNIT SERVICE HOURS	See "Hours Generator On-Line."

INDEX TO LICENSED UNITS

ARKANSAS 1 A-3
ARKANSAS 2 A-5
BEAVER VALLEY 1 B-3
BIG ROCK POINT 1 B-5
BROWNS FERRY 1 B-7
BROWNS FERRY 2 B-9
BROWNS FERRY 3 B-11
BRUNSWICK 1 B-13
BRUNSWICK 2 B-15
CALVERT CLIFFS 1 C-3
CALVERT CLIFFS 2 C-5
COOK 1 C-7
COOK 2 C-9
COOPER STATION C-11
CRYSTAL RIVER 3 C-13
DAVIS-BESSE 1 D-3
DRESDEN 1 D-5
DRESDEN 2 D-7
DRESDEN 3 D-9
DUANE ARNOLD D-11
FARLEY 1 F-3
FITZPATRICK F-5
FORT CALHOUN 1 F-7
FORT ST VRAIN F-9
GINNA G-3
HADDAM NECK H-3
HATCH 1 H-5
HATCH 2 H-7
HUMBOLDT BAY H-9
INDIAN POINT 1 I-3
INDIAN POINT 2 I-5
INDIAN POINT 3 I-7
KEWAUNEE K-3
LA CROSSE L-3
MAINE YANKEE M-3
MILLSTONE 1 M-5
MILLSTONE 2 M-7
MONTICELLO M-9
NINE MILE POINT 1 N-3
NORTH ANNA 1 N-5
OCONEE 1 O-3
OCONEE 2 O-5
OCONEE 3 O-7
OYSTER CREEK 1 O-9
PALISADES P-3
PEACH BOTTOM 2 P-5
PEACH BOTTOM 3 P-7
PILGRIM 1 P-9
POINT BEACH 1 P-11
POINT BEACH 2 P-13
PRAIRIE ISLAND 1 P-15
PRAIRIE ISLAND 2 P-17
QUAD CITIES 1 Q-3
QUAD CITIES 2 Q-5
RANCHO SECO 1 R-3
ROBINSON 2 R-5
SALEM 1 S-3
SAN ONOFRE 1 S-5
ST LUCIE 1 S-7
SURRY 1 S-9
SURRY 2 S-11
THREE MILE ISLAND 1 T-3
THREE MILE ISLAND 2 T-5
TROJAN T-7
TURKEY POINT 3 T-9
TURKEY POINT 4 T-11
VERMONT YANKEE 1 V-3
YANKEE-ROWE 1 Y-3
ZION 1 Z-3
ZION 2 Z-5

SECTION 1 – CURRENT DATA SUMMARIES

MONTHLY HIGHLIGHTS OF COMMERCIAL NUCLEAR POWER UNITS

FOR MONTH OF

JUNE 1980

LICENSED POWER REACTORS:

NUMBER		CAPACITY [1] MWe (Net)
67	IN COMMERCIAL OPERATION	49,010
67 [2]	LICENSED TO OPERATE	49,010
3 [3]	LICENSED FOR FUEL LOADING AND LOW POWER TESTING	

POWER GENERATION	REPORT MONTH	PREVIOUS MONTH	YEAR-TO-DATE
1. GROSS ELECTRICAL (MWHE)	19,331,405	19,313,241	118,987,817
2. NET ELECTRICAL (MWHE)	18,363,856	18,384,016	113,333,148
3. AVG. UNIT SERVICE FACTOR (%)	59.2	57.0	59.2
4. AVERAGE UNIT AVAILABILITY FACTOR (%)	59.2	57.0	59.4
5. AVERAGE UNIT CAPACITY FACTOR (MDC) (%)	52.1	48.8	52.4
6. AVG. UNIT CAPACITY FACTOR (DER) (%)	50.6	47.6	51.1
7. FORCED OUTAGE RATE (%)	12.3	10.6	11.0

NOTES

[1] BASED UPON MAXIMUM DEPENDABLE CAPACITY; DESIGN ELECTRIC RATING USED IF MDC NOT DETERMINED.

[2] EXCLUDES HUMBOLDT BAY (DER = 65), AND THREE MILE ISLAND 2 (DER = 906) WHICH HAVE OPERATING LICENSES BUT ARE SHUTDOWN INDEFINITELY AND INDIAN POINT 1 WHICH IS TO BE DECOMMISSIONED.

[3] SEQUOYAH 1 (DER =1148) WAS LICENSED ON FEBRUARY 29, 1980
NORTH ANNA 2 (DER = 907) WAS LICENSED ON APRIL 11, 1980
SALEM 2 (DER =1115) WAS LICENSED ON APRIL 18, 1980

ACTUAL VS. POTENTIAL ENERGY PRODUCTION

			% OF POTENTIAL PRODUCTION
POTENTIAL ENERGY PRODUCTION IN THIS PERIOD BY UNITS IN COMMERCIAL OPERATION (Using Maximum Dependable Capacity Net)		35,304,480 MWHe	
ENERGY ACTUALLY PRODUCED DURING THIS REPORT PERIOD	=	18,363,856 MWHe NET	52.0
ENERGY NOT PRODUCED DURING THIS REPORT PERIOD DUE TO SCHEDULED OUTAGES (NET)	=	10,472,972 MWHe	29.7
ENERGY NOT PRODUCED DURING THIS REPORT PERIOD DUE TO FORCED OUTAGES (NET)	=	4,085,776 MWHe	11.6
ENERGY NOT PRODUCED DURING THIS REPORT PERIOD FOR OTHER REASONS (NET)	=	2,381,875 MWHe	6.7
TOTAL		35,304,480 MWHe =	100%
ENERGY NOT PRODUCED THIS REPORT PERIOD DUE TO NRC IMPOSED SHUTDOWNS AND RESTRICTIONS	=	1,921,642 MWHe	5.4
NUMBER OF UNITS WITH POWER RESTRICTIONS IMPOSED BY NRC	=	1	

OUTAGE DATA

	NUMBER	HOURS	PERCENT OF CLOCK TIME	MWHE LOST PRODUCTION
1. FORCED OUTAGES DURING MONTH	65	5,206.7	10.8	4,085,776
2. SCHEDULED OUTAGES DURING THE MONTH	41	14,435.7	29.9	10,472,972
3. TOTAL	106	19,642.4	40.7	14,558,749

MWHE LOST PRODUCTION = Down time x maximum dependable capacity net

DERATED UNITS

UNIT NAME	MAXIMUM DEPENDABLE CAPACITY (MWe - NET)	POWER LIMIT (MWe - NET)
COOPER STATION	784	635
†FORT ST. VRAIN	330	231
INDIAN POINT 2	849	824
INDIAN POINT 3	965	917
POINT BEACH 1	495	390
ROBINSON 2	2300 MWt	2200 MWt
SURRY 1	775	704

†Restricted by NRC.

MONTHLY HIGHLIGHTS OF COMMERCIAL NUCLEAR POWER UNITS

FOR MONTH OF

JUNE 1980

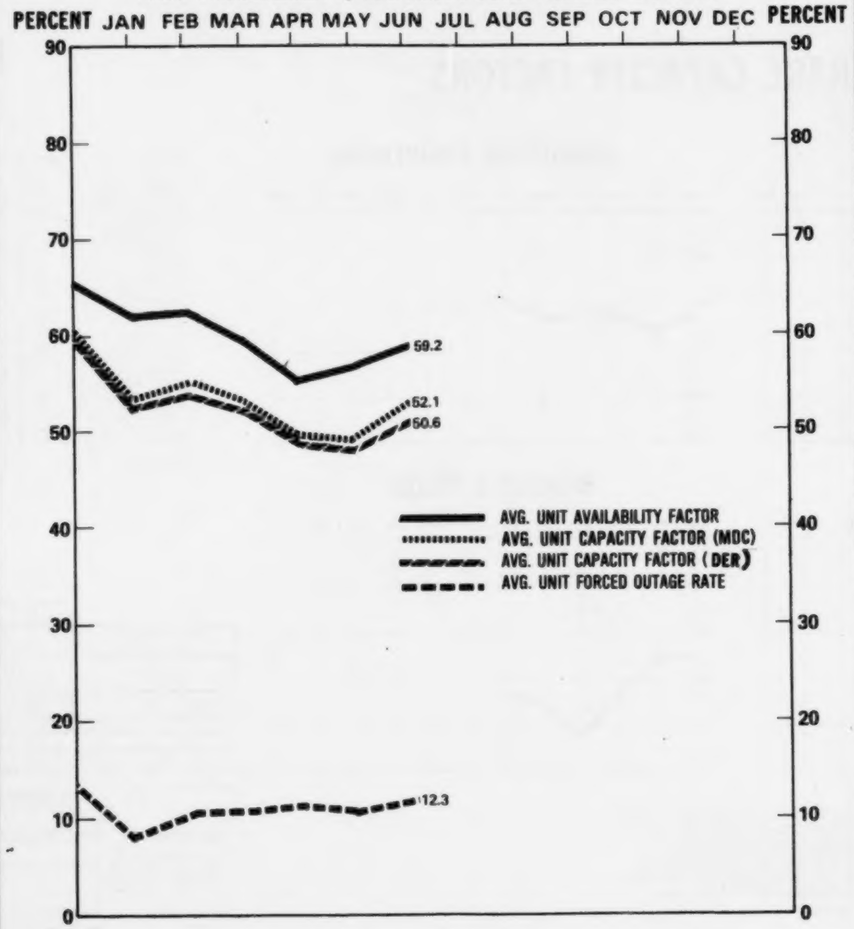
REASONS FOR SHUTDOWN

REASON	NUMBER	TOTAL HOURS LOST
A - Equipment Failure	50	4,275.8
B - Maintenance or Test	19	1,583.1
C - Refueling	15	9,334.4
D - NRC Restriction	7	2,313.7
E - Operator Training and License Examination	1	24.2
F - Administrative	0	0
G - Operational Error	2	10.3
H - Other	12	2,100.9
TOTAL	106	19,642.4

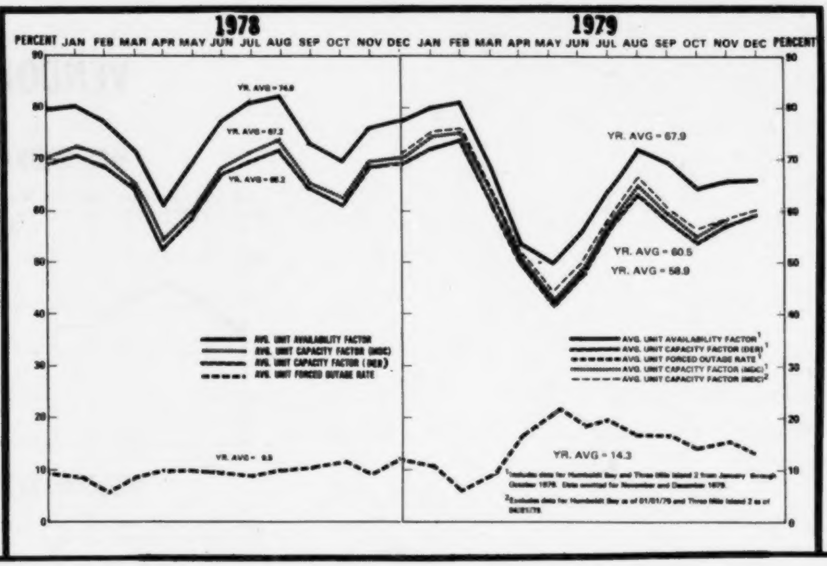
SHUTDOWNS GREATER THAN 72 HOURS EACH

UNIT NAME	REASON	UNIT NAME	REASON
ARKANSAS 1	A	KEWAUNEE	C
ARKANSAS 2	2A	LA CROSSE	A
BEAVER VALLEY 1	H	MAINE YANKEE	A
BROWNS FERRY 2	B	MILLSTONE 1	4A
BRUNSWICK 1	C	MILLSTONE 2	D
BRUNSWICK 2	C	NORTH ANNA 1	B
COOK 1	C	OCONEE 1	D
COOK 2	H	OCONEE 2	B
COOPER STATION	C	OCONEE 3	A
CRYSTAL RIVER 3	C	OYSTER CREEK 1	C
DAVIS–BESSE 1	C	PEACH BOTTOM 2	C
DRESDEN 1	H	SAN ONOFRE 1	C
FARLEY 1	B	ST LUCIE 1	A
FITZPATRICK	C	SURRY 2	D
FORT CALHOUN 1	C	THREE MILE ISLAND 1	D
FORT ST VRAIN	H	TROJAN	C,D
HADDAM NECK	C	TURKEY POINT 4	B
HATCH 1	3A	YANKEE ROWE 1	A
INDIAN POINT 2	A	ZION 2	C

UNIT AVAILABILITY, CAPACITY AND FORCED OUTAGE



UNIT AVAILABILITY, CAPACITY AND FORCED OUTAGE



[1] Includes data for Humboldt Bay and Three Mile Island 2 from January through October 1979. Data omitted for November and December 1979.

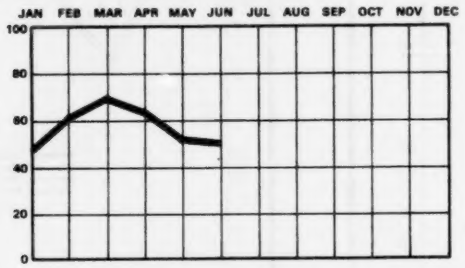
[2] Excludes data for Humboldt Bay as of 01/01/79 and Three Mile Island 2 as of 04/01/79.

REFUELING SCHEDULE

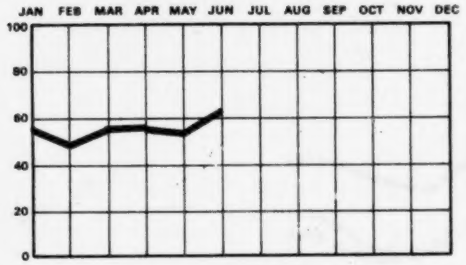
Please refer to the table on page 3-2 for the scheduled refueling date for each nuclear powerplant in this publication.

VENDOR AVERAGE CAPACITY FACTORS

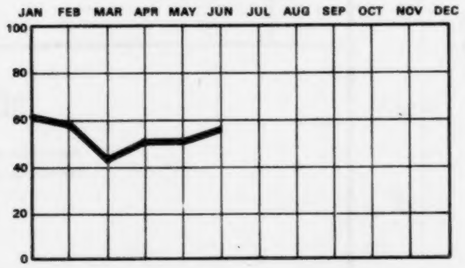
WESTINGHOUSE



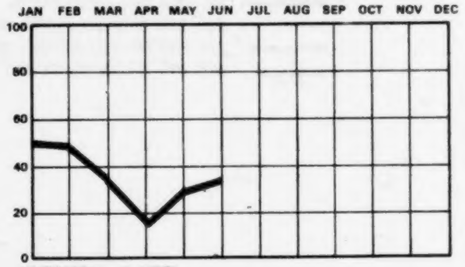
COMBUSTION ENGINEERING



GENERAL ELECTRIC



BABCOCK & WILCOX



NOTE: THIS DISPLAY OF AVERAGE CAPACITY FACTORS PROVIDES A GENERAL PERFORMANCE COMPARISON OF PLANTS SUPPLIED BY THE FOUR NUCLEAR STEAM SUPPLY SYSTEM VENDORS. ONE MUST BE CAREFUL WHEN DRAWING CONCLUSIONS REGARDING THE REASONS FOR THE PERFORMANCE LEVELS INDICATED, SINCE PLANT PERFORMANCE MAY BE AFFECTED BY UNSPECIFIED FACTORS SUCH AS: (1) VARIOUS PLANT DESIGNS AND MODELS ARE INCLUDED FOR EACH VENDOR; (2) TURBINE/GENERATORS AND (3) DIFFERENT ARCHITECT/ENGINEERS ARE ALSO INVOLVED.

JUNE 1980

OPERATIONS

GE BWR's	CFMDC
BROWNS FERRY 1	78.2
BROWNS FERRY 2	74.2
BROWNS FERRY 3	85.4
BRUNSWICK 1	.0
BRUNSWICK 2	.0
COOPER STATION	57.4
DRESDEN 1	.0
DRESDEN 2	79.3
DRESDEN 3	97.6
DUANE ARNOLD	89.3
FITZPATRICK	.0
HATCH 1	30.4
HATCH 2	72.6
MILLSTONE 1	15.3
MONTICELLO	99.1
NINE MILE POINT 1	88.6
OYSTER CREEK 1	.0
PEACH BOTTOM 2	.0
PEACH BOTTOM 3	90.4
PILGRIM 1	83.1
QUAD CITIES 1	61.5
QUAD CITIES 2	91.3
VERMONT YANKEE 1	82.3

23 UNITS.

W PWR's	
BEAVER VALLEY 1	.0
COOK 1	.0
COOK 2	87.6
FARLEY 1	40.7
GINNA	101.9
HADDAM NECK	.0
INDIAN POINT 2	61.3
INDIAN POINT 3	79.8
KEWAUNEE	10.3
NORTH ANNA 1	57.3
POINT BEACH 1	74.9
POINT BEACH 2	96.7
PRAIRIE ISLAND 1	87.9
PRAIRIE ISLAND 2	82.1
ROBINSON 2	91.3
SALEM 1	82.5
SAN ONOFRE 1	.0
SURRY 1	86.0
SURRY 2	.0
TROJAN	.0
TURKEY POINT 3	97.1
TURKEY POINT 4	42.5
YANKEE-ROWE 1	.0
ZION 1	79.9
ZION 2	.0

25 UNITS

CE PWR's	CFMDC
ARKANSAS 2	59.1
CALVERT CLIFFS 1	94.2
CALVERT CLIFFS 2	98.2
FORT CALHOUN 1	47.0
MAINE YANKEE	56.7
MILLSTONE 2	20.2
PALISADES	102.9
ST LUCIE 1	33.6

8 UNITS

B&W PWR's	CFMDC
ARKANSAS 1	62.5
CRYSTAL RIVER 3	.0
DAVIS-BESSE 1	.0
OCONEE 1	63.4
OCONEE 2	7.7
OCONEE 3	45.9
RANCHO SECO 1	96.6
THREE MILE ISLAND 1	.0

8 UNITS

	GE BWR's	W PWR's	CE PWR's	B&W PWR's	All PWR's
NET ELECTRICAL PRODUCTION	7,079,122	6,654,188	2,790,930	1,708,352	11,153,470
MDC NET	17,351	18,413	6,090	6,737	31,240
CFMDC	56.7	50.2	63.7	35.2	49.6

Capacity factor in this page, denoted as CFMDC, is a function of the net maximum dependable capacity. See the corresponding definition in the Glossary. The vendor averages are computed by the formula:

$$\frac{\text{Net Electrical Energy Produced by Vendor}}{\text{Potential Electrical Production By Vendor In This Month.}} \text{ X } 100\%$$

Units Excluded	Reason for Exclusion
Humboldt Bay Three Mile Island 2	Unit Shutdown Indefinitely
Big Rock Point	Not Representative of Typical BWRs
LaCrosse	Not Representative of Typical BWRs and Atypical Vendor
Fort St. Vrain	Atypical Vendor and Reactor Design

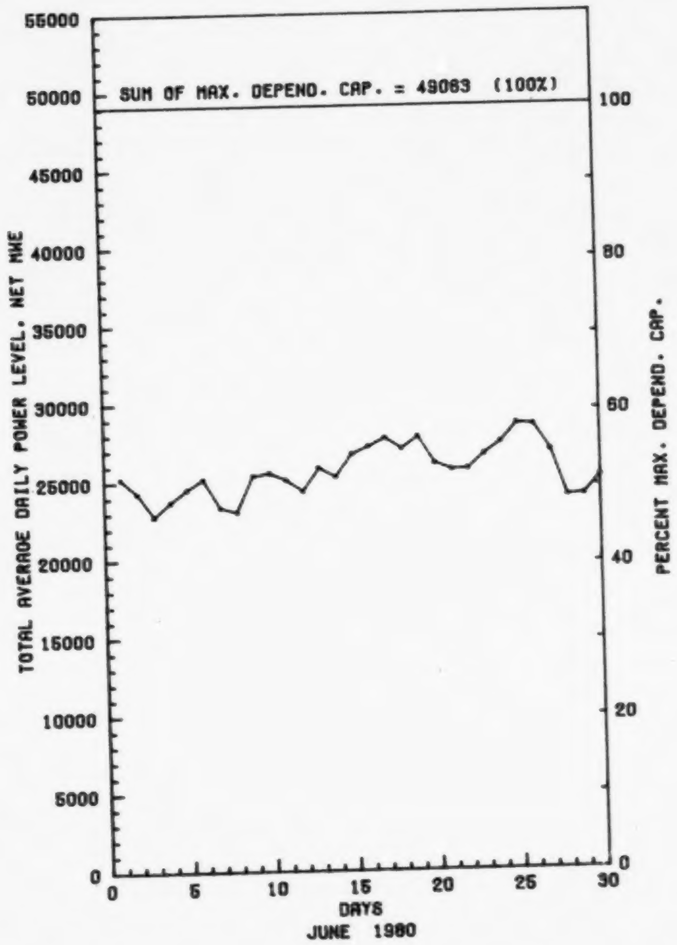
AVERAGE DAILY POWER LEVEL FOR ALL COMMERCIALLY OPERATING UNITS

This chart depicts the average daily power level for the units in commercial operation during the month.

The straight line on the graph labelled "SUM OF MAX. DEPEND. CAP." is plotted at the value shown by summing the separate maximum dependable capacities of the commercially operating units (in Net MWe). The plot shown below the line is calculated by summing the separate average daily power levels of the commercially operating units for each day of the month.

The scale on the left vertical axis runs in 1,000 MWe increments from 0 to 55,000 MWe (Net). The right vertical axis shows the percentage in 10% increments, up to 100% of the "SUM OF MAX. DEPEND. CAP."

It should be recognized that the 100% line would be obtainable only if all of the commercially operating units operated at 100% capacity, 24 hours per day, for the entire month. In other words, since any power generator must occasionally shut down to refuel and/or perform needed maintenance, and also since 100% capacity production is not always required by power demands, the 100% line is a theoretical goal and not a practical one.



ERRATA

Corrections and Amendments to Previously-Reported Data

(NOTE: These changes are reflected in the data contained in the current report)

CORRECTIONS TO NUREG-0020 VOL. 4 NO. 6 JUNE 1980

Page	Unit Name	Item	Correction
	NONE		

MULTIPLE - ISSUE CORRECTIONS

DAVIS BESSE 1

THE STATION HAS DETERMINED THAT ITS MAXIMUM DEPENDABLE CAPACITY IS 890 MWe. PRIOR TO THIS DETERMINATION BY THE UTILITY, WE HAVE BEEN USING THE DER OF 906 MWe TO DETERMINE ITEM 21. UNIT CAPACITY FACTOR (MDC NET.) THE MONTHLY VALUE OF ITEM 21. BASED ON A MDC OF 890 ARE:

11/77	25.4%
12/77	40.9%
1/78	37.8%
2/78	54.6%
3/78	54.9%
4/78	56.9%
8/78	19.5%
9/78	76.2%
10/78	14.0%
11/78	66.1%
12/78	26.2%
1/79	38.2%
2/79	76.3%
3/79	76.1%
7/79	57.7%
8/79	96.9%
9/79	74.6%
10/79	44.3%
11/79	18.9%
1/80	75.0%

FARLEY 1

ITEM 18. NET ELECTRICAL ENERGY HAS BEEN REVISED BY THE UTILITY TO INCLUDE PERIODS OF NEGATIVE GENERATION (RATHER THAN LISTING "O"). THE UPDATED MONTHLY VALUES BASED ON NEGATIVE GENERATION WOULD BE:

3/79	109,926
4/79	– 3,786
5/79	– 5,014
6/79	– 5,802
7/79	– 6,158
8/79	– 5,796
9/79	– 5,422
10/79	–12,194
11/79	120,550
12/79	541,920
1/80	526,908
2/80	187,626
3/80	561,330
4/80	571,758
5/80	583,600

TOTAL 10,425,238

SECTION 2 – OPERATING POWER REACTORS

ARKANSAS 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Arkansas/Pope 6 miles NW of Russellville
2.	Electric Reliability Council	Southwest Power Pool
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2568
5.	Design Electrical Rating (MWE Net)	850
6.	Date of Initial Criticality	August 6, 1974
7.	Date of Commercial Operation	December 19, 1974
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Dardenalle Reservoir

Utility & Contractor Information

10.	Licensee	Arkansas Power and Light Company
11.	Corporate Address	P.O. Box 551 Little Rock, Arkansas 72203
12.	Corporate Contact	W. Cavanaugh III Vice Pres., Generation & Construction
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	B&W
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	IV
17.	IE Branch Chief	G. L. Madsen
18.	IE Section Chief	D. M. Hunnicutt
19.	IE Resident Inspector	W. D. Johnson (501) 968-3290
20.	Licensing Project Manager	G. S. Vissing
21.	Docket Number	50-313
22.	License Number/Date of Issue	DPR-51, 5/21/74
23.	Public Document Room	Arkansas Polytechnic College Russellville, Arkansas 72801

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

FACILITY ITEMS (PLANS AND PROCEDURES):

MANAGERIAL ITEMS

PLANT STATUS:

AT POWER OPERATIONS.

LAST IE SITE INSPECTION DATE: MAY 27 - 30, 1980

INSPECTION REPORT NO: 50-313/80-09

SUMMARY

INSPECTION SUMMARY

INSPECTION DURING APRIL 22 - MAY 21, 1980 (80-07): ROUTINE, ANNOUNCED INSPECTION INCLUDING CONFIRMATORY ORDER FOLLOW-UP, FOLLOW-UP ON IE BULLETIN 79-27, FOLLOW-UP ON OPEN ITEM, OPERATIONAL SAFETY VERIFICATION, AND REACTOR COOLANT PUMP SEAL FAILURE. NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION DURING MAY 27 - 30, 1980 (80-09): ROUTINE, UNANNOUNCED INSPECTION OF DESIGN CHANGES AND MODIFICATION PROGRAM, AND TEST AND EXPERIMENT PROGRAM. NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

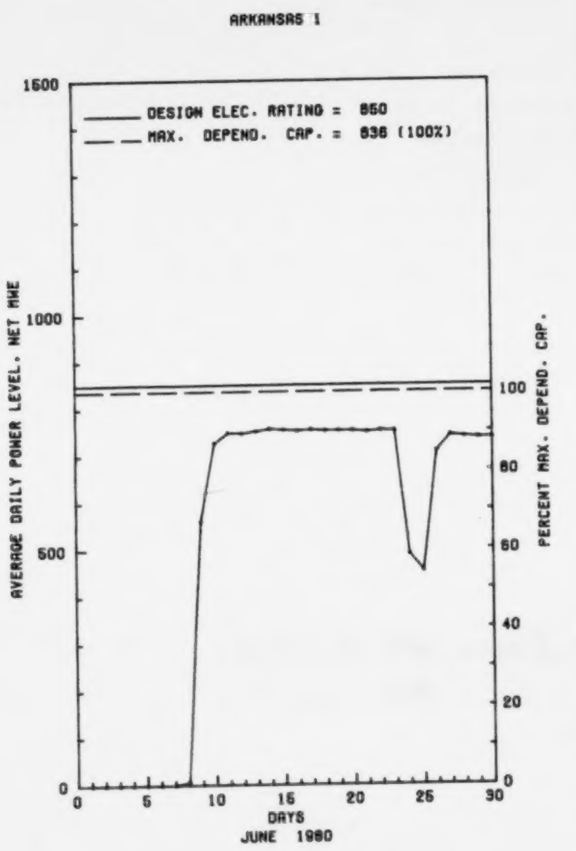
```
=========================================================================================
NUMBER   DATE OF   DATE OF                        SUBJECT
          EVENT     REPORT
=========================================================================================
80-17    5/19/80   6/13/80      REACTOR BUILDING VENTILATION BYPASS DAMPER INOPERABLE
03L-0
80-18    5/24/80   6/5/80       WELD DEFECT IN CONTAINMENT FLUED HEAD PENETRATION
01T-0
=========================================================================================
```

REVIEWED BY OMPA LOEB: Lobe **DATE:** 8/4/80

REVIEWED BY NRR DL: **DATE:** 8/5/80

ARKANSAS 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-313
2. Utility Contact: L. S. Bramlett (501) 968-2519
3. Licensed Thermal Power (MWt): 2568
4. Nameplate Rating (Gross MWe): 1003 x 0.9 = 903
5. Design Electrical Rating (Net MWe): 850
6. Maximum Dependable Capacity (Gross MWe): 883
7. Maximum Dependable Capacity (Net MWe): 836
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	4,367.0	43,498.0
12. Hours Reactor Was Critical	527.7	2,238.1	32,376.3
13. Reactor Reserve Shutdown Hours	64.0	528.8	4,776.2
14. Hours Generator On-Line	520.1	2,191.7	31,662.2
15. Unit Reserve Shutdown Hours	.0	20.8	817.5
16. Gross Thermal Energy (MWH)	1,238,916	5,133,611	76,395,655
17. Gross Electrical Energy (MWH)	395,446	1,594,371	25,315,527
18. Net Electrical Energy (MWH)	376,493	1,516,755	24,150,791
19. Unit Service Factor	72.2	50.2	65.3
20. Unit Availability Factor	72.2	50.7	67.0
21. Unit Capacity Factor (MDC Net)	62.5	41.5	59.6
22. Unit Capacity Factor (DER Net)	61.5	40.9	58.6
23. Unit Forced Outage Rate	27.8	28.0	16.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-05	05/10/80	F	198.1	A	4	80-015/01T	CA	PUMPXX	"C" RCP SEAL FAILURE.
80-06	06/24/80	F	9.8	H	3	80-022/01T	ZZ	ZZZZZZ	PARTIAL LOSS OF OFFSITE POWER.

SUMMARY

THE UNIT RETURNED TO SERVICE ON 06/08/80 AFTER RCS SEAL REPAIR AND OPERATED NEAR FULL POWER THE REST OF THE MONTH, EXCEPT FOR AN OUTAGE ON 06/24/80 DUE TO PARTIAL OFFSITE POWER LOSS.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

ARKANSAS 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Arkansas/Pope 6 miles NW of Russellville
2.	Electric Reliability Council	Southwest Power Pool
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2815
5.	Design Electrical Rating (MWE Net)	912
6.	Date of Initial Criticality	12-5-78
7.	Date of Commercial Operation	3/26/80
8.	Condenser Cooling Method	Recirculating
9.	Condenser Cooling Water Source	Cooling Tower

Utility & Contractor Information

10.	Licensee	Arkansas Power & Light Company
11.	Corporate Address	P.O. Box 551 Little Rock, Arkansas 72203
12.	Corporate Contact	W. Cavanaugh III Vice Pres., Generation & Construction
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Combustion Engineering, Inc.
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	IV
17.	IE Branch Chief	G. L. Madsen
18.	IE Section Chief	D. M. Hunnicutt
19.	IE Resident Inspector	W. D. Johnson (501) 968-3290
20.	Licensing Project Manager	R. E. Martin 492-8087
21.	Docket Number	50-368
22.	License Number/Date of Issue	NFP-6, 9/1/78
23.	Public Document Room	Arkansas Polytechnic College Russellville, Arkansas 72801

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO TECHNICAL SPEC 6.8.1, THE LICENSEE FAILED TO FOLLOW STEP 4.20 OF PROCEDURE 1004.14, "JOB ORDERS" IN THAT THE JUNCTION BOX COVERS WERE NOT REPLACED ON A HPSI CONTROL VALVE PRIOR TO DOCUMENTING COMPLETION OF ELECTRICAL MAINTENANCE ON THE VALVE AND RETURNING THE VALVE TO OPERATION.
(8007 INFRACTION)

CONTRARY TO TECH SPEC 6.8.1, THE LICENSEE FAILED TO FOLLOW THE REQUIREMENTS OF PROCEDURE 1005.04 IN THAT: 1. ENTRIES HAD NOT BEEN CLEARED IN LOG AFTER JUMPER/BYPASS HAD BEEN REMOVED. 2. DUPLICATE TAG NUMBERS WERE ISSUED FOR TWO SEPARATE TAGOUTS. 3. COMPLETED PAGES OF THE LOG HAD NOT BEEN FORWARDED TO THE NUC. PLANT ADMINISTRATOR FOR FILING.
(8009 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

FACILITY ITEMS (PLANS AND PROCEDURES):

MANAGERIAL ITEMS:

PLANT STATUS:

AT THE END OF THIS PERIOD, THE UNIT WAS IN AN OUTAGE FOR REACTOR COOLANT PUMP SEAL REPLACEMENT.

LAST IE INSPECTION DATE: MAY 27 - 30, 1980

INSPECTION REPORT NO: 50-368/80-09

SUMMARY

INSPECTION SUMMARY

INSPECTION DURING APRIL 22 - MAY 21, 1980 (80-07): ROUTINE, ANNOUNCED INSPECTION INCLUDING FOLLOW-UP ON IE BULLETIN 79-27, LICENSEE REVIEW OF POWER ASCENSION TEST RESULTS, FOLLOW-UP ON LICENSEE EVENT REPORTS, INSPECTOR REVIEW OF POWER ASCENSION TEST DATA, FOLLOW-UP ON OPEN ITEM, ISOTHERMAL TEMPERATURE AND POWER COEFFICIENTS OF REACTIVITY, CORE POWER DISTRIBUTION OPERATIONAL SAFETY VERIFICATION, AND SURVEILLANCE OBSERVATION. WITHIN THE NINE AREAS INSPECTED, ONE ITEM OF NONCOMPLIANCE WAS IDENTIFIED (INFRACTION - ADHERENCE TO PROCEDURE).

INSPECTION DURING MAY 27 - 30, 1980 (80-09): ROUTINE, UNANNOUNCED INSPECTION OF DESIGN CHANGES AND MODIFICATION PROGRAM, AND TEST AND EXPERIMENT PROGRAM. WITHIN THE TWO AREAS INSPECTED, ONE APPARENT ITEM OF NONCOMPLIANCE WAS NOTED (INFRACTION - FAILURE TO ADHERE TO PROCEDURES).

REPORTS RECEIVED FROM LICENSEE

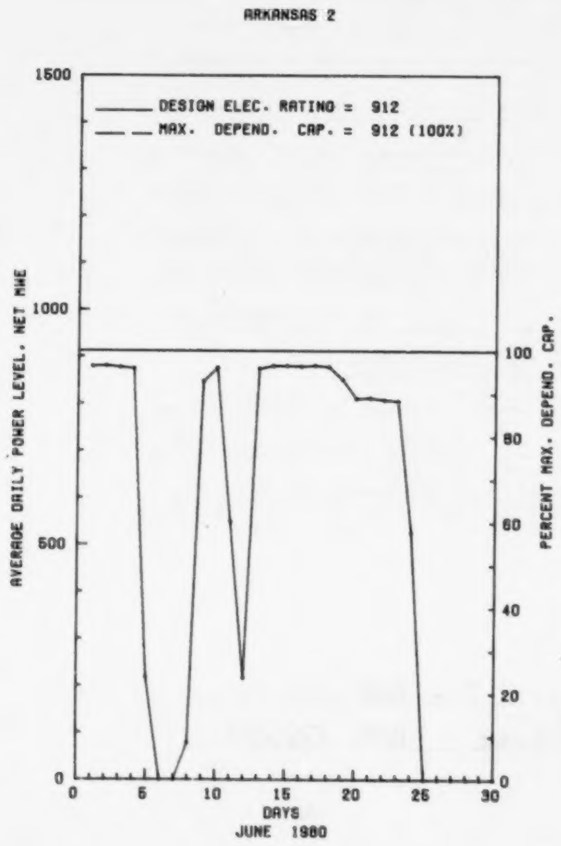
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-23 03L-0	5/6/80	5/29/80	SODIUM HYDROXIDE TANK DISCHARGE VALVE INOPERABLE
80-25 03X-1	5/7/80	6/27/80	SODIUM HYDROXIDE TANK DISCHARGE VALVE INOPERABLE
80-26 03L-0	5/14/80	6/11/80	CONTROL ROOM AIR CONDITIONER
80-27 03L-0	5/16/80	6/12/80	SERVICE WATER SYSTEM HANGER INOPERABLE
80-28 03L-0	5/16/80	6/12/80	PLANT PROTECTIVE SYSTEM POWER SUPPLY FAILURE
80-29 03L-0	5/20/80	6/16/80	DIESEL GENERATOR DELUGE SYSTEM INOPERABLE
80-30 03L-0	5/20/80	6/16/80	STEAM DRIVEN EMERGENCY FEEDWATER PUMP TRIP
80-31 04L-0	5/21/80	6/25/80	CONDENSER GAS SAMPLE MONITOR INOPERABLE
80-37 01T-0	6/3/80	6/17/80	SAFETY INJECTION TANK BORON CONCENTRATION LOW

REVIEWED BY OMPA LOEB: MR Beebe **DATE:** 08/04/80

REVIEWED BY NRR DL: R Martin **DATE:** 8/5/80

ARKANSAS 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-368
2. Utility Contact: Rex Pendergraft (501) 968-2519
3. Licensed Thermal Power (MWt): 2815
4. Nameplate Rating (Gross MWe): 959
5. Design Electrical Rating (Net MWe): 912
6. Maximum Dependable Capacity (Gross MWe): 912
7. Maximum Dependable Capacity (Net MWe): 912
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11 Hours in Reporting Period	720.0	2,327.0	2,327.0
12. Hours Reactor Was Critical	485.3	1,762.3	1,762.3
13. Reactor Reserve Shutdown Hours	.0	80.9	80.9
14. Hours Generator On-Line	478.5	1,729.1	1,729.1
15. Unit Reserve Shutdown Hours	.0	75.0	75.0
16. Gross Thermal Energy (MWH)	1,254,162	4,142,818	4,142,818
17. Gross Electrical Energy (MWH)	406,950	1,356,980	1,356,980
18. Net Electrical Energy (MWH)	388,381	1,291,646	1,291,646
19. Unit Service Factor	66.5	74.3	74.3
20. Unit Availability Factor	66.5	77.5	77.5
21. Unit Capacity Factor (MDC Net)	59.1	60.9	60.9
22. Unit Capacity Factor (DER Net)	59.1	60.9	60.9
23. Unit Forced Outage Rate	33.5	25.7	25.7

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 07/02/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-11	06/05/80	F	75.7	A	1		HB	PIPEXX	STEAM LEAK ON MAIN STEAM LINE IN REACTOR BUILDING. UNIT WAS COOLED DOWN AND LEAK REPAIRED.
80-12	06/11/80	F	13.5	A	3		IA	INSTRU	TWO SPURIOUS DNBR/LPD TRIPS OCCURRED SIMULTANEOUSLY ON THE CORE PROTECTION CALCULATORS (CPC'S).
80-13	06/24/80	F	152.3	A	3	80-042/01T	EA	ELECON	PARTIAL LOSS OF OFFSITE POWER DUE TO A GROUND FAULT ON ONE OF THE 500KV TRANSMISSION LINES.

SUMMARY

THE PLANT RAN AT NEAR FULL POWER EXCEPT FOR OUTAGES ON 06/05/80 BECAUSE OF A STEAM LEAK, ON 06/11/80 DUE TO A GROUND SHIELD MALFUNCTION AND ON 06/24/80 WHEN SOME OFFSITE POWER WAS LOST. THE UNIT REMAINED OFFLINE FOR RCP SEAL REPAIR.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

NRC POWER RESTRICTIONS: NONE

BEAVER VALLEY 1

FACILITY DATA

Facility Description

1. Location (state/county) (nearest population center)	Pennsylvania/Beaver 5 miles E of E. Liverpool, Ohio
2. Electric Reliability Council	East Central Area Reliability Coordination Agreement
3. Type of Reactor	PWR
4. Power Level (MWT)	2660
5. Design Electrical Rating (MWE Net)	852
6. Date of Initial Criticality	May 10, 1976
7. Date of Commercial Operation	10/01/76 (50%) 04/30/77 (100%)
8. Condenser Cooling Method	Cooling tower/Closed loop
9. Condenser Cooling Water Source	Ohio River

Utility & Contractor Information

10. Licensee	Duquesne Light Company
11. Corporate Address	435 Sixth Avenue Pittsburgh, Pennsylvania 15219
12. Corporate Contact	C. N. Dunn, Vice President
13. Architect/Engineer	Stone & Webster
14. Nuclear Steam Supply System	Westinghouse
15. Constructor	Stone & Webster

Regulatory Information

16. IE Region responsible	I
17. IE Branch Chief	E. J. Brunner
18. IE Section Chief	E. C. MCCABE
19. IE Resident Inspector	D. A. BECKMAN (412-643-9200/9201)
20. Licensing Project Manager	W. J. Ross
21. Docket Number	50-334
22. License Number/Date of Issue	DPR-66, 1/30/76
23. Public Document Room	B. F. Jones Memorial Library 633 Franklin Avenue Alquippa, PA 15001

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

FACILITY ITEMS (PLANS AND PROCEDURES):

- Refueling/modification outage commenced 11/30/79; outage is expected to continue until at least 7/22/80.

PLANT STATUS:

- In cold shutdown. Major modifications are underway on the following safety-related systems: Bergen-Patterson large bore snubbers; containment spray pumps; refueling water storage tank; low head safety injection pumps; pipe hangers, support, and base plate inspection; reactor coolant pump motors; auxiliary feedwater pumps; and low pressure turbine rotors.
- LAST IE SITE INSPECTION DATE: 6/30/80
- INSPECTION REPORT NO: 50-334/80-19

SUMMARY

INSPECTION SUMMARY

- No reports issued.

REPORTS RECEIVED FROM LICENSEE

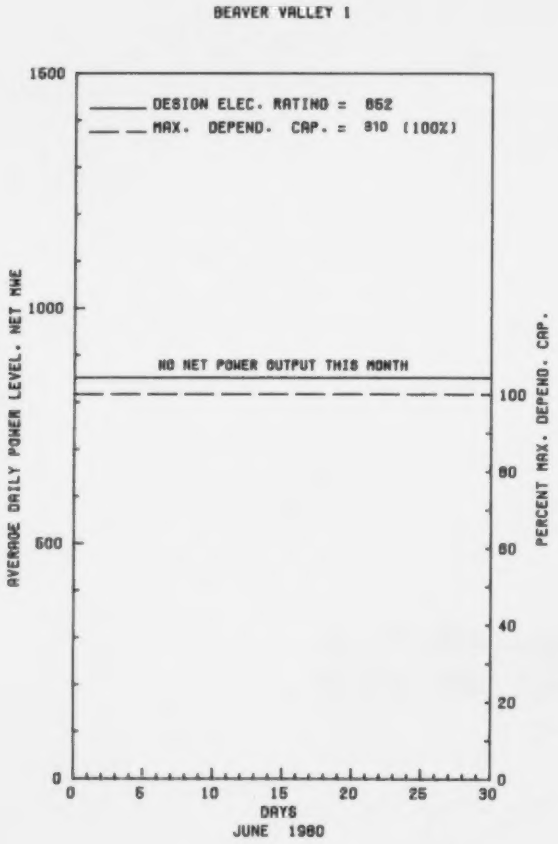
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-0039P	05/15/80	06/09/80	Met computer moved from Pgh to site. Certain output resistors not removed after reinstallation caused faulted signals. Resistors removed. System calibrated satisfactory.
80-003/01T	01/24/80	06/02/80	Component cooling water restraint H-95 seismic inadequacy resulted in alteration of support to a 3-way restraint.
80-026/03L	04/22/80	05/21/80	Unplanned liquid discharge occurred due to improper valve alignment and effluent valve seat leakage. No discharge limits were exceeded as a result of the discharge.
80-027/03L	04/30/80	05/28/80	Failure of EDG cooling water check valve had potential to degrade EDG cooling capability. Valves overhauled. Licensee to inspect other similar valves.
80-028/03L	05/04/80	06/02/80	Power lost to No. 4 vital bus inverter due to blown main power supply fuse. Plant was in mode 5 and opposite train inverter in service.
80-030/01T	05/19/80	06/02/80	Diesel fire pump oos due overheating. Electric fire pump oos. Portable diesel fire pump hooked into fire main with a second portable pump to be hooked up until repair effected.
80-031/01T	05/21/80	06/04/80	RHR high point vent nipple leak isolation caused RHR loss due false high press signal from inst loop de-energization.
80-032/03L	05/03/80	06/02/80	VCT outlet valve MOV-CH-115C was found to be receiving continuous shut signal as a result of short in level control circuit. Short corrected. Remaining module wire insected ok.
80-034/03L	05/12/80	06/09/80	Half the fire nozzles in one main filter bank found plugged with rust and unidentified matorial. Nozzles and piping cleaned. Other MFB nozzles will be testod.
80-035/03L	05/12/80	06/09/80	Surveillance test identified fuel bldg internal pressure less than 1/8 in. water column relative atmos. flow director damper (friction type) had drifted shut. Damper removed.
80-036/03L	05/14/80	06/09/80	Operators noted build-up on RH-211 weld fitting. Fitting ground out and rewelded. Support methods being studied. Other RHR train had been available.
80-038/03L	05/17/80	06/16/80	Gaseous waste tank GW-TK-1A found leaking. Pressure decrease of flow 18 to 6 psig. No activity in tank. Leak was on flanges of flow orifice PT for gaseous waste surge tank.
80-040/01T	05/03/80	06/04/80	Re-review of radiographs revealed code rejectable welds on SI and Aux feed lines. Defective welds were repaired. Future radiographs to be checked by two level II examiners.

REVIEWED BY OMPA LOEB: Philip A Ross **DATE:** 08/04/80

REVIEWED BY NRR DL: WJ Ross **DATE:** 8/5/80

BEAVER VALLEY 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-334
2. Utility Contact: DAVID R. TIMKO (412) 643-5308
3. Licensed Thermal Power (MWt): 2660
4. Nameplate Rating (Gross MWe): 1026 x 0.9 = 923
5. Design Electrical Rating (Net MWe): 852
6. Maximum Dependable Capacity (Gross MWe): 845
7. Maximum Dependable Capacity (Net MWe): 810
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	4,367.0	36,527.0
12. Hours Reactor Was Critical	.0	.0	13,774.7
13. Reactor Reserve Shutdown Hours	.0	.0	4,482.7
14. Hours Generator On-Line	.0	.0	13,105.9
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	0	26,974,253
17. Gross Electrical Energy (MWH)	0	0	8,277,949
18. Net Electrical Energy (MWH)	0	0	7,570,743
19. Unit Service Factor	.0	.0	37.5
20. Unit Availability Factor	.0	.0	37.5
21. Unit Capacity Factor (MDC Net)	.0	.0	28.3
22. Unit Capacity Factor (DER Net)	.0	.0	27.8
23. Unit Forced Outage Rate	.0	.0	46.4

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): MAJOR MODIFICATIONS OUTAGE (DECEMBER, 1979 - JULY, 1980).
25. If Shut Down At End of Report Period, Estimated Date of Startup: 07/22/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
1	01/01/80	S	720.0	D	4		ZZ	ZZZZZZ	UNIT SHUTDOWN FOR MAJOR MODIFICATIONS AS REQUIRED BY THE NUCLEAR REGULATORY COMMISSION, INCLUDING NRC BULLETINS IEB 79-02 AND 79-14.

SUMMARY

THE PLANT REMAINS SHUTDOWN FOR AN NRC ORDERED MODIFICATION OUTAGE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

BIG ROCK POINT 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Michigan/Charlevoix 4 miles NE of Charlevoix
2.	Electric Reliability Council	East Central Area Reliability Coordination Agreement
3.	Type of Reactor	BWR
4.	Power Level (MWT)	240
5.	Design Electrical Rating (MWE Net)	72
6.	Date of Initial Criticality	September 27, 1962
7.	Date of Commercial Operation	March 29, 1963
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10.	Licensee	Consumers Power Company
11.	Corporate Address	212 West Michigan Avenue Jackson, Michigan 49201
12.	Corporate Contact	R. B. DeWitt
13.	Architect/Engineer	Bechtel Corporation
14.	Nuclear Steam Supply System	GE
15.	Constructor	Bechtel Corporation

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	D. C. BOYD
19.	IE Resident Inspector	G. Wright
20.	Licensing Project Manager	W. A. Paulson
21.	Docket Number	50-155
22.	License Number/Date of Issue	DPR-6 issued 8/30/62
23.	Public Document Room	Charlevoix Public Library 107 Clinton Street Charlevoix, Michigan

INSPECTION STATUS

ENFORCEMENT STATUS

THE LICENSEE DID NOT CONDUCT PROPER PERSONNEL SCREENING IN ACCORDANCE WITH SECURITY PLAN COMMITMENTS.
(8005 DEFICIENCY)

THE LICENSEE FAILED TO PROVIDE ADEQUATE KEY CONTROL.
(8005 DEFICIENCY)

THE LICENSEE FAILED TO PROVIDE ADEQUATE ASSESSMENT AIDS THAT CONFORM WITH APPROVED SECURITY PLAN COMMITMENTS.
(8005 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENTS

+ A leakage path continues to preclude isolation of the offgas system. Administrative guidelines have been imposed to shutdown the reactor well before reaching the technical specification effluent limits. (80-04)

FACILITY ITEMS

+ Weaknesses were identified in health physics technician and radiation worker training, personal contamination control practices, procedure coverage and adheronce, high radiation area access controls, and extendible probe survey instrument supplies. (80-04)

MANAGERIAL ITEMS

+ Weaknesses were identified in health physics professional and technical level staffing, offshift radiation protection coverage, definition of the health physics group's authority over radiological work activities, and ALARA formalization. (80-04)

PLANT STATUS

Operating routinely.

+LAST IE SITE INSPECTION DATE: March 3-14, 1980

+ INSPECTION REPORT NO: 80-04

SUMMARY

INSPECTION SUMMARY

+ Inspection on March 3-14, (80-04): Special, announced inspection of health physics program, including organization and management, training, quality assurance, procedures, internal and external exposure controls, surveys and access controls, instrumentation, ALARA, radioactive waste, facilities and equipment, and accident response capabilities. The inspection involved 350 inspector hours onsite by five NRC inspectors. Several significant weaknesses in the health physics program were identified. These weaknesses are in the areas of staffing, training, procedures, radiological controls, instrumentation, ALARA and radwaste. Two apparent items of noncompliance were found (inadequate offshift radiation protection coverage; and inadequate high radiation area access controls).

REPORTS RECEIVED FROM LICENSEE

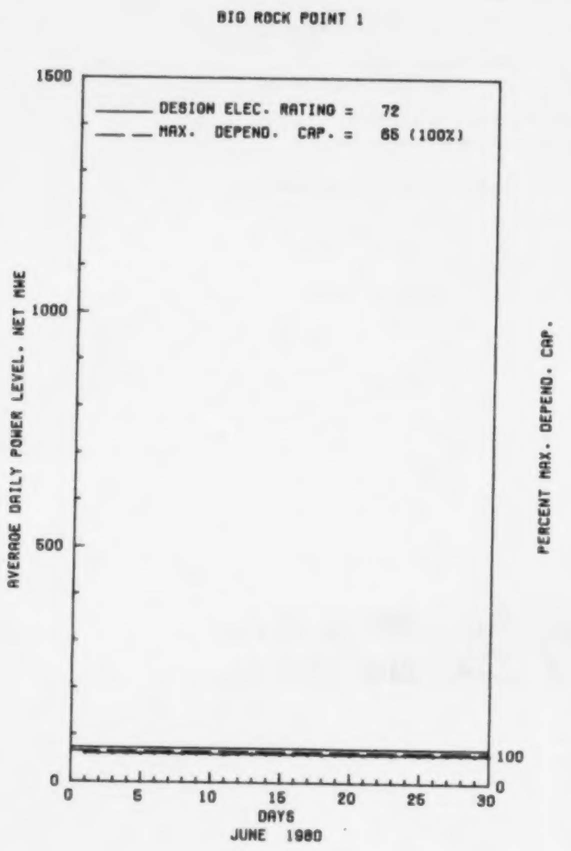
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-09/ 03L-0	04/15/80	05/19/80	Leakage tests of equipment and presonnel locks.
80-11/ 03L-0	05/01/80	05/30/80	Reactor depressurization system.
80-14/ 03L-0	05/12/80	06/11/80	Following intermittent auto. test circ. alarms, the sensor channel D of the Reac. Depressurizing System was removed from service to correct the problem.
80-15/ 03L-0	05/15/80	06/11/80	Following intermittent auto. test circ. alarms, the sensor channel D was removed from service.
80-16/ 03L-0	05/27/80	06/11/80	Sensor Channel D of the Reactor Depressurizing System was removed from service.

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 8-4-80

REVIEWED BY NRR DL: W.A. Paulson **DATE:** 8-5-80

BIG ROCK POINT 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-155
2. Utility Contact: G. Himebauch (616) 547-6537 x180
3. Licensed Thermal Power (MWt): 240
4. Nameplate Rating (Gross MWe): 70.6 x 0.85 = 60
5. Design Electrical Rating (Net MWe): 72
6. Maximum Dependable Capacity (Gross MWe): 69
7. Maximum Dependable Capacity (Net MWe): 65
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

ITEM 21 CALCULATED WITH A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	4,367.0	151,290.0
12. Hours Reactor Was Critical	720.0	4,045.2	104,178.2
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	720.0	4,032.5	102,031.4
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	151,280	846,446	18,564,377
17. Gross Electrical Energy (MWH)	46,113	258,109	5,494,018
18. Net Electrical Energy (MWH)	43,593	243,907	5,672,244
19. Unit Service Factor	100.0	92.3	67.4
20. Unit Availability Factor	100.0	92.3	67.4
21. Unit Capacity Factor (MDC Net)	93.1	85.9	55.4
22. Unit Capacity Factor (DER Net)	84.1	77.6	52.1
23. Unit Forced Outage Rate	.0	1.0	23.7

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	NONE								

SUMMARY

THE UNIT OPERATED NEAR FULL POWER THE ENTIRE REPORT PERIOD.

LEGEND

[1] Type:
F–Forced
S–Scheduled

[2] Reason:
A–Equipment Failure
B–Maintenance or Test
C–Refueling
D–Regulatory Restriction
E–Operator Training & License Examination
F–Administrative
G–Operational Error
H–Other

[3] Method:
1–Manual
2–Manual Scram
3–Automatic Scram
4–Continuations
5–Load Reduction
9–Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

BROWNS FERRY 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Alabama/Limestone 10 miles NW of Decatur
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	3293
5.	Design Electrical Rating (MWE Net)	1065
6.	Date of Initial Criticality	August 17, 1973
7.	Date of Commercial Operation	August 1, 1974
8.	Condenser Cooling Method	Once Through With Forced Draft Cooling Towers
9.	Condenser Cooling Water Source	Tennessee River

Utility & Contractor Information

10.	Licensee	Tennessee Valley Authority
11.	Corporate Address	500A Chestnut Street Tower II Chattanooga, Tennessee 37401
12.	Corporate Contact	H. G. Parris, Manager of Power.
13.	Architect/Engineer	TVA
14.	Nuclear Steam Supply System	GE
15.	Constructor	TVA

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. Lewis
18.	IE Section Chief	H. C. Dance
19.	IE Resident Inspectur	R. Sullivan (205) 729-6196
20.	Licensing Project Manager	R. Clark
21.	Docket Number	50-259
22.	License Number/Date of Issue	DPR-33, 6/26/73
23.	Public Document Room	Athens Public Library South and Forrest Athens, Alabama 35611

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO 10CFR71.5 THE LICENSEE DID NOT COMPLY WITH DOT REGULATIONS DURING A SHIPMENT OF RADIOACTIVE MATERIAL. THE DOSE RATE ON THE SIDE OF A CLOSED VEHICLE WAS MEASURED TO BE 600 MREM/HR. 49CFR173.393(J)(2) SPECIFIES A MAXIMUM DOSE RATE OF 200 MREM/HR.
(8021 VIOLATION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

100 PERCENT FULL POWER OPERATION.

+ LAST IE SITE INSPECTION DATE: MAY 1-23, 1980

+ INSPECTION REPORT NO: 50-259/80-21

SUMMARY

INSPECTION SUMMARY

+ INSPECTION APRIL 28 - MAY 1 (80-21): EACH LICENSEE-SHIPPER VEHICLE WAS INSPECTED FOR COMPLIANCE WITH DEPARTMENT OF TRANSPORTATION (DOT) AND NUCLEAR REGULATORY COMMISSION (NRC) REGULATIONS AS FOLLOWS: (1) SHIPPING PAPER REQUIREMENTS; (2) DOT PLACARDING REQUIREMENTS; (3) RADIATION LEVELS; AND (4) DOT MARKING AND LABELING REQUIREMENTS FOR PACKAGES. OF THE FOUR AREAS INSPECTED INVOLVING THIS SHIPMENT, ONE APPARENT ITEM OF NONCOMPLIANCE WAS IDENTIFIED (SEVERITY LEVEL I - RADIATION LEVELS ON SURFACE OF CLOSED EXCLUSIVE USE VEHICLE IN EXCESS OF REGULATORY LIMITS).

+ INSPECTION MAY 19-23 (80-22): INCLUDED REVIEW OF SECURITY PLAN; SECURITY ORGANIZATION; SECURITY PROGRAM AUDIT; RECORDS AND REPORTS; TESTING AND MAINTENANCE; LOCKS AND KEY CONTROL; PHYSICAL BARRIERS (PROTECTED AND VITAL AREAS); POWER SUPPLY; LIGHTING; ASSESSMENT AIDS; ACCESS CONTROLS (SEARCH, ESCORT, VITAL AREAS); DETECTION AIDS; COMMUNICATIONS. THE INSPECTION INVOLVED 27 INSPECTOR HOURS ON SITE BY TWO NRC INSPECTORS. TEN INSPECTION HOURS WERE ACCOMPLISHED DURING OFFSHIFT PERIODS. OF THE 16 AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE WERE IDENTIFIED IN 12 AREAS; FOUR ITEMS OF NONCOMPLIANCE WERE IDENTIFIED IN THE OTHER FOUR AREAS: (1) PHYSICAL BARRIERS - VITAL AREA BARRIER INCOMPLETE; (2) KEY AND LOCK CONTROL - INADEQUATE INSTALLATION OF LOCK; (3) ASSESSMENT AIDS - CCTV SURVEILLANCE CAPABILITY NOT REDUNDANT; (4) ACCESS CONTROL - VEHICLE ESCORT NOT EQUIPPED WITH PORTABLE RADIO.

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-015/ 01T-0	02/06/80	06/09/80	CONTAINMENT DESIGN CRITERIA NOT MET.
80-031	04/15/80	05/14/80	A FAULT ON NORMAL FEEDER BUS RESULTED IN TRIPPING THE 480V SHUTDOWN BOARD 1B.
80-032/ 03L-0	04/14/80	05/13/80	RADIATION MONITOR 1-RM-90-250 WAS FOUND TO BE ERRATIC AND DECLARED INOPERABLE.
80-033	04/17/80	05/15/80	480 VOLT REACTOR MOV BOARD TRIPPED DUE TO SHORT CIRCUIT.
80-034	04/20/80	05/19/80	REACTOR WATER CLEAN UP ISOLATION VALVES CLOSED DUE TO LOOSE WIRE ON RELAY COIL.
80-035	04/24/80	05/22/80	LOW OIL LEVEL ALARM 1B RECIRCULATION PUMP MOTOR BEARING.
80-036	04/22/80	05/21/80	1B RECIRCULATION MG SET TRIPPED DURING TRANSFER OPERATIONS.
80-038	04/30/80	05/29/80	HALOGEN MONITOR CHANNEL INOPERABLE.
80-039	05/07/80	05/20/80	CONTAINMENT AIR MONITORING SYSTEM HYDROGEN SENSORS NOT ENVIRONMENTALLY QUALIFIED.
80-040	05/06/80	06/04/80	DRYWELL SAMPLING SYSTEM INOPERABLE DUE TO VALVE MALFUNCTION.
80-041	05/08/80	06/05/80	SMOKE DETECTOR ALARM WOULD NOT CLEAR.
80-042	05/12/80	06/10/80	CORE SPRAY ROOM COOLER FAILED SURVEILLANCE.
80-043	04/11/80	05/23/80	GASKET LEAKAGE RESIDUAL HEAT REMOVAL HEAT EXCHANGER 1A DURING SURVEILLANCE TEST.

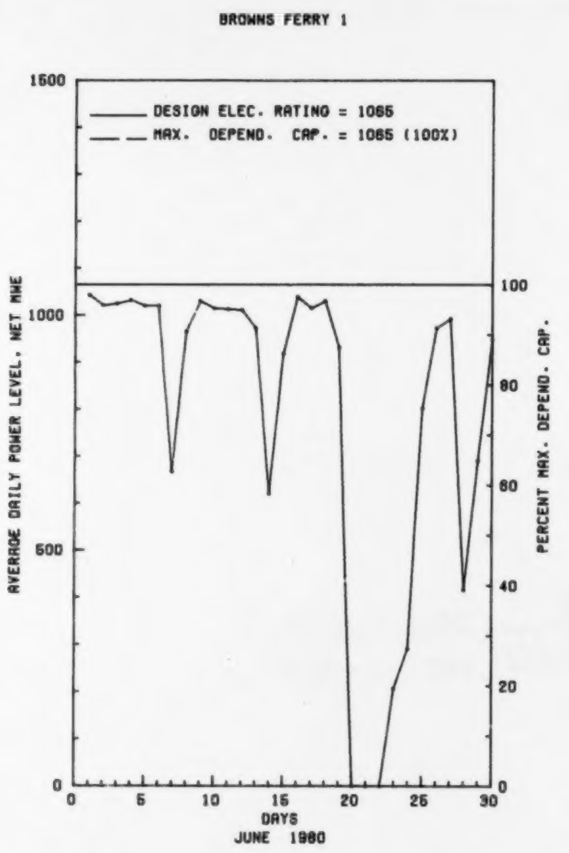
REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 8-4-80

REVIEWED BY NRR DL: Dick Clark **DATE:** 08-05-80

GPO 871-087

BROWNS FERRY 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-259
2. Utility Contact: Don Green (205) 729-6846
3. Licensed Thermal Power (MWt): 3293
4. Nameplate Rating (Gross MWe): 1280 x 0.9 = 1152
5. Design Electrical Rating (Net MWe): 1065
6. Maximum Dependable Capacity (Gross MWe): 1098
7. Maximum Dependable Capacity (Net MWe): 1065
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	4,367.0	51,865.0
12. Hours Reactor Was Critical	659.8	2,191.6	30,468.7
13. Reactor Reserve Shutdown Hours	60.2	344.0	5,033.5
14. Hours Generator On-Line	643.6	2,072.1	29,685.0
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,974,667	5,791,462	79,927,955
17. Gross Electrical Energy (MWH)	615,870	1,869,260	26,387,720
18. Net Electrical Energy (MWH)	599,539	1,812,746	25,614,716
19. Unit Service Factor	89.4	47.4	57.2
20. Unit Availability Factor	89.4	47.4	57.2
21. Unit Capacity Factor (MDC Net)	78.2	39.0	46.4
22. Unit Capacity Factor (DER Net)	78.2	39.0	46.4
23. Unit Forced Outage Rate	10.6	16.4	32.1

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
126	06/01/80	S	.0	B	5				DERATED TO REPLACE BRUSHES IN "B" RECIRCULATION PUMP M.G. SET.
127	06/07/80	S	.0	B	5				DERATED FOR TURBINE CONTROL VALVE TESTS AND SI'S.
128	06/17/80	F	10.0	A	3				MOISTURE SEPARATOR HIGH LEVEL.
129	06/17/80	F	36.8	A	2				"B" RECIRCULATION PUMP LOW OIL LEVEL ALARM.
130	06/21/80	S	.0	B	5				DERATED FOR TURBINE CONTROL VALVE TESTS AND SI'S.
131	06/23/80	F	8.1	A	2				MAINTENANCE TO EHC OIL LEAK.
132	06/24/80	F	11.6	B	3				WHILE PERFORMING CONTROL VALVE CLOSURE SI.
133	06/24/80	F	10.0	B	2				WHILE PERFORMING CONTROL VALVE CLOSURE SI.
134	06/28/80	F	.0	A	5				DERATED DUE TO MAIN STEAM LINE HIGH TEMPERATURE.

SUMMARY

THE UNIT OPERATED NEAR FULL POWER EXCEPT FOR NUMEROUS OUTAGES AND FIVE SCRAMS (LISTED ABOVE).

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

BROWNS FERRY 2

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Alabama/Limestone 10 miles NW of Decatur
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	3293
5.	Design Electrical Rating (MWE Net)	1065
6.	Date of Initial Criticality	July 20, 1974
7.	Date of Commercial Operation	March 1, 1975
8.	Condenser Cooling Method	Once Through With Forced Draft Cooling Towers
9.	Condenser Cooling Water Source	Tennessee River

Utility & Contractor Information

10.	Licensee	Tennessee Valley Authority
11.	Corporate Address	500A Chestnut Street Tower II Chattanooga, Tennessee 37401
12.	Corporate Contact	H. G. Parris, Manager of Power
13.	Architect/Engineer	TVA
14.	Nuclear Steam Supply System	GE
15.	Constructor	TVA

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. Lewis
18.	IE Section Chief	H. C. Dance
19.	IE Resident Inspector	R. Sullivan (205) 729-6196
20.	Licensing Project Manager	R. Clark
21.	Docket Number	50-260
22.	License Number/Date of Issue	DPR-52, 6/28/74
23.	Public Document Room	Athens Public Library South and Forrest Athens, Alabama 35611

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

100 PERCENT FULL POWER OPERATION.

+ LAST IE SITE INSPECTION DATE: MAY 19-23, 1980

+ INSPECTION REPORT NO: 50-260/80-16

SUMMARY

INSPECTION SUMMARY

+ INSPECTION MAY 19-23 (80-16): INCLUDED REVIEW OF SECURITY PLAN: SECURITY ORGANIZATION; SECURITY PROGRAM AUDIT; RECORDS AND REPORTS; TESTING AND MAINTENANCE; LOCKS AND KEY CONTROL; PHYSICAL BARRIERS (PROTECTED AND VITAL AREAS); POWER SUPPLY; LIGHTING; ASSESSMENT AIDS; ACCESS CONTROLS (SEARCH, ESCORT, VITAL AREAS); DETECTION AIDS; COMMUNICATIONS. THE INSPECTION INVOLVED 27 INSPECTOR HOURS ON SITE BY TWO NRC INSPECTORS. TWENTY INSPECTION HOURS WERE ACCOMPLISHED DURING OFFSHIFT PERIODS. OF THE 16 AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE WERE IDENTIFIED IN 12 AREAS; FOUR ITEMS OF NONCOMPLIANCE WERE IDENTIFIED IN THE OTHER FOUR AREAS: (1) PHYSICAL BARRIERS - VITAL AREA BARRIER INCOMPLETE; (2) KEY AND LOCK CONTROL - INADEQUATE INSTALLATION OF LOCK; (3) ASSESSMENT AIDS - CCTV SURVEILLANCE CAPABILITY NOT REDUNDANT; (4) ACCESS CONTROL - VEHICLE ESCORT NOT EQUIPPED WITH PORTABLE RADIO.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

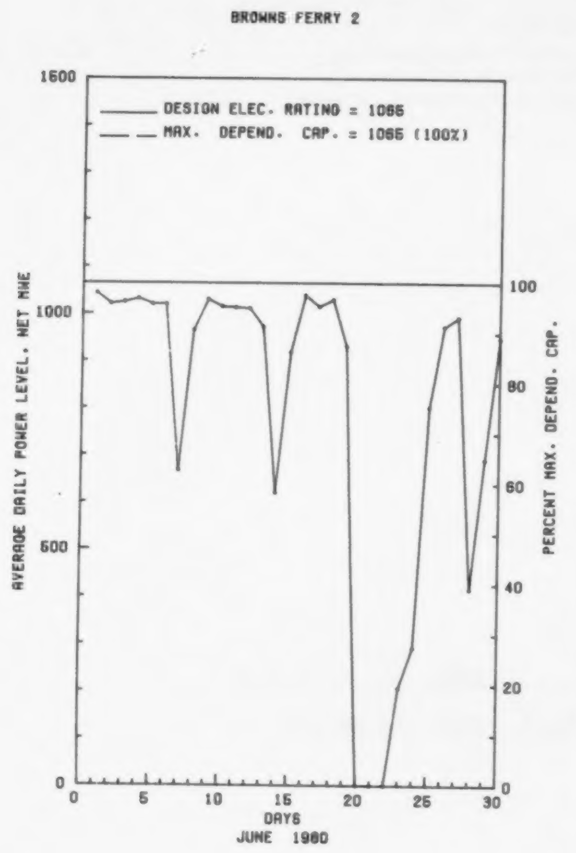
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-020	04/21/80	05/20/80	REACTOR HIGH PRESSURE SWITCH SETPOINT DRIFT.
80-021	05/06/80	06/04/80	PARTICULATE CHANNEL RECORDER READING ERRATIC DECLARED INOPERABLE.

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 8-4-80

REVIEWED BY NRR DL: Dick Clark **DATE:** 08-05-80

BROWNS FERRY 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-260
2. Utility Contact: Don Green (205) 729-6846
3. Licensed Thermal Power (MWt): 3293
4. Nameplate Rating (Gross MWe): 1280 x 0.9 = 1152
5. Design Electrical Rating (Net MWe): 1065
6. Maximum Dependable Capacity (Gross MWe): 1098
7. Maximum Dependable Capacity (Net MWe): 1065
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	4,367.0	46,776.0
12. Hours Reactor Was Critical	641.4	3,788.0	28,272.4
13. Reactor Reserve Shutdown Hours	78.6	579.0	12,353.9
14. Hours Generator On-Line	611.4	3,679.1	27,343.6
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,776,475	10,557,875	76,712,875
17. Gross Electrical Energy (MWH)	585,310	3,522,000	25,472,068
18. Net Electrical Energy (MWH)	568,875	3,426,932	24,758,401
19. Unit Service Factor	84.9	84.2	58.5
20. Unit Availability Factor	84.9	84.2	58.5
21. Unit Capacity Factor (MDC Net)	74.2	73.7	49.7
22. Unit Capacity Factor (DER Net)	74.2	73.7	49.7
23. Unit Forced Outage Rate	3.4	14.0	34.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
REFUEL OUTAGE SEPTEMBER, 1980.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
140	06/07/80	F	.0	A	5				DERATED DUE TO "A" RECIRCULATION PUMP TRIP.
141	06/13/80	S	.0	H	5				DERATED FOR CONTROL ROD SEQUENCE EXCHANGE FROM "A" TO "B"
142	06/19/80	S	87.2	B	2				SCHEDULED MAINTENANCE.
143	06/24/80	F	11.0	B	3				CONDENSER LOW VACUUM DURING MAINTENANCE ON CONDENSER DRAIN VALVE.
144	06/28/80	F	10.4	B	3				CONDENSER LOW VACUUM.

SUMMARY

THE UNIT RAN NEAR FULL POWER WITH REDUCTIONS DUE TO PUMP TRIP AND ROD SEQUENCE EXCHANGES. OUTAGES OCCURRED ON 06/24/80 AND ON 06/28/80 BECAUSE OF LOW CONDENSER VACUUM, AND ON 06/19/80 FOR SCHEDULED MAINTENANCE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

BROWNS FERRY 3

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Alabama/Limestone 10 miles NW of Decatur
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	3293
5.	Design Electrical Rating (MWE Net)	1065
6.	Date of Initial Criticality	August 8, 1976
7.	Date of Commercial Operation	March 1, 1977
8.	Condenser Cooling Method	Once Through With Forced Draft Cooling Tower
9.	Condenser Cooling Water Source	Tennessee River

Utility & Contractor Information

10.	Licensee	Tennessee Valley Authority
11.	Corporate Address	500A Chestnut Street Tower II Chattanooga, Tennessee 37401
12.	Corporate Contact	H. G. Parris, Manager of Power
13.	Architect/Engineer	TVA
14.	Nuclear Steam Supply System	GE
15.	Constructor	TVA

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R.C. Lewis
18.	IE Section Chief	H. C. Dance
19.	IE Resident Inspector	R. Sullivan (205) 729-6196
20.	Licensing Project Manager	R. Clark
21.	Docket Number	50-296
22.	License Number/Date of Issue	DPR-68, 7/2/76
23.	Public Document Room	Athens Public Library South and Forrest Athens, Alabama 35611

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

FULL POWER OPERATIONS.

+ LAST IE SITE INSPECTION DATE: MAY 19-23, 1980

+ INSPECTION REPORT NO: 50-296/80-17

SUMMARY

INSPECTION SUMMARY

+ INSPECTION MAY 19-23 (80-17): INCLUDED REVIEW OF SECURITY PLAN; SECURITY ORGANIZATION; SECURITY PROGRAM AUDIT; RECORDS AND REPORTS; TESTING AND MAINTENANCE; LOCKS AND KEY CONTROL; PHYSICAL BARRIERS (PROTECTED AND VITAL AREAS); POWER SUPPLY; LIGHTING; ASSESSMENT AIDS; ACCESS CONTROLS (SEARCH, ESCORT, VITAL AREAS); DETECTION AIDS; COMMUNICATIONS. THE INSPECTION INVOLVED 26 INSPECTOR HOURS ON SITE BY TWO NRC INSPECTORS. TWENTY INSPECTION HOURS WERE ACCOMPLISHED DURING OFFSHIFT PERIODS. OF THE 16 AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE WERE IDENTIFIED IN 12 AREAS; FOUR ITEMS OF NONCOMPLIANCE WERE IDENTIFIED IN THE OTHER FOUR AREAS: (1) PHYSICAL BARRIERS - VITAL AREA BARRIER INCOMPLETE; (2) KEY AND LOCK CONTROL - INADEQUATE INSTALLATION OF LOCK; (3) ASSESSMENT AIDS - CCTV SURVEILLANCE CAPABILITY NOT REDUNDANT; (4) ACCESS CONTROL - VEHICLE ESCORT NOT EQUIPPED WITH PORTABLE RADIO.

REPORTS RECEIVED FROM LICENSEE

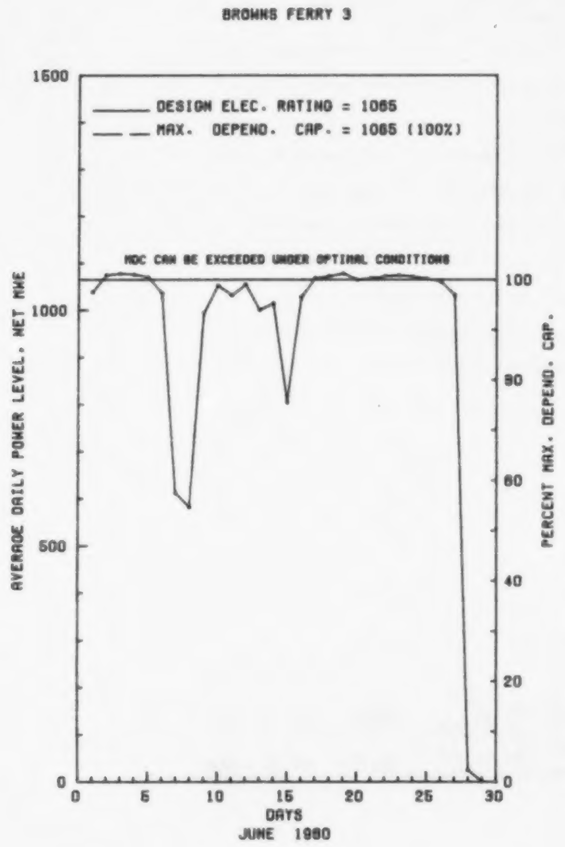
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-010	04/10/80	05/05/80	YARWAY LIMIT SWITCHES SET POINT DRIFT.
80-011	04/17/80	05/15/80	REACTOR WATER CLEAN UP ISOLATION VALVE FAIL TO CLOSE WHEN OPERATED NORMALLY.
80-012	05/09/80	06/06/80	YARWAY LIMIT SWITCH FAILED SURVEILLANCE TEST.
80-014	05/12/80	06/10/80	REACTOR CORE ISOLATION COOLING TRIPPED ON OVERSPEED DURING TEST.
80-016	05/13/80	06/11/80	DIESEL GENERATOR HEAT EXCHANGER COOLING WATER FLOW INADEQUATE.
80-017	04/04/80	06/02/80	MISSING PAGE FROM A PROCEDURE RESULTS IN NO SURVEILLANCE TEST ON FIRE SYSTEM IN ONE ZONE.

REVIEWED BY OMPA LOEB: J. A. Rizzo **DATE:** 08-04-80

REVIEWED BY NRR DL: Dick Clark **DATE:** 08-05-80

BROWNS FERRY 3

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-296
2. Utility Contact: Don Green (205) 729-6846
3. Licensed Thermal Power (MWt): 3293
4. Nameplate Rating (Gross MWe): 1280 x 0.9 = 1152
5. Design Electrical Rating (Net MWe): 1065
6. Maximum Dependable Capacity (Gross MWe): 1098
7. Maximum Dependable Capacity (Net MWe): 1065
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	4,367.0	29,231.0
12. Hours Reactor Was Critical	638.9	3,853.0	22,761.5
13. Reactor Reserve Shutdown Hours	91.2	101.7	1,760.1
14. Hours Generator On-Line	636.4	3,822.2	22,260.6
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,990,334	11,537,761	65,045,627
17. Gross Electrical Energy (MWH)	672,640	4,029,030	21,631,610
18. Net Electrical Energy (MWH)	654,737	3,927,577	20,815,350
19. Unit Service Factor	88.4	87.5	76.2
20. Unit Availability Factor	88.4	87.5	76.2
21. Unit Capacity Factor (MDC Net)	85.4	84.6	66.9
22. Unit Capacity Factor (DER Net)	85.4	84.6	66.9
23. Unit Forced Outage Rate	11.6	12.5	9.6

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUEL OUTAGE OCTOBER, 1980
25. If Shut Down At End of Report Period, Estimated Date of Startup: 07/12/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
129	06/06/80	F	.0	A	5				DERATED DUE TO "B" RECIRCULATION PUMP RUNBACK.
130	06/07/80	F	13.1	A	3				GROUND PROTECTION RELAY TRIP.
131	06/14/80	S	.0	B	5				DERATED FOR TURBINE CONTROL VALVE TESTS, SI'S AND CONTROL ROD ADJUSTMENT.
132	06/27/80	F	70.5	A	2				MAINTENANCE TO FEEDWATER PIPING.

SUMMARY

THE PLANT OPERATED NEAR FULL POWER WITH REDUCTIONS ON 06/06/80 FOR RECIRC PUMP RUN BACK, AND ON 06/14/80 FOR VALVE TESTING. SCRAMS OCCURRED ON 06/07/80 DUE TO LOAD REDUCTION AND ON 06/28/80 FOR FEEDWATER LEAKAGE. DURING THIS SCRAM ABOUT ONE THIRD OF THE CONTROL RODS DID NOT ENTER THE CORE IMMEDIATELY; THE PROBLEM WILL BE FURTHER RESEARCHED.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

NRC POWER RESTRICTIONS: NONE

BRUNSWICK 1

FACILITY DATA

Facility Description

1. Location (state/county) (nearest population center)	North Carolina/Brunswick 3 miles N of Southport
2. Electric Reliability Council	Southeastern Electric Reliability Council
3. Type of Reactor	BWR
4. Power Level (MWT)	2436
5. Design Electrical Rating (MWE Net)	821
6. Date of Initial Criticality	October 8, 1976
7. Date of Commercial Operation	March 18, 1977
8. Condenser Cooling Method	Once Through
9. Condenser Cooling Water Source	Cape Fear River

Utility & Contractor Information

10. Licensee	Carolina Power and Light Company
11. Corporate Address	411 Fayetteville Street Raleigh, North Carolina 27602
12. Corporate Contact	J. A. Jones Executive Vice President
13. Architect/Engineer	United Engineers & Constructors, Inc.
14. Nuclear Steam Supply System	GE
15. Constructor	Brown and Root, Inc.

Regulatory Information

16. IE Region responsible	II
17. IE Branch Chief	R.C. Lewis
18. IE Section Chief	P.J. Kellogg
19. IE Resident Inspector	J. E. Outzs (919) 457-9631
20. Licensing Project Manager	J. Hannon
21. Docket Number	50-325
22. License Number/Date of Issue	DPR-71, 9/8/76
23. Public Document Room	Southport-Brunswick County Library 109 W. Moore Street Southport, North Carolina 28461

INSPECTION STATUS

ENFORCEMENT STATUS

AS REQUIRED BY THE CP&L QUALITY ASSURANCE MANUAL, THE LICENSEE IS COMMITTED TO ANSI 18.7-1972. SECTION 5.3.5(3) OF ANSI 18.7-1972 STATES THAT INSTRUCTIONS SHALL BE INCLUDED OR REFERENCED IN PROCEDURES FOR RETURNING EQUIPMENT TO ITS NORMAL OPERATING STATUS. PT 15.1, STANDBY GAS TREATMENT SYSTEM, PT15.2.1, HEPA FILTER EFFICIENCY AND PT 15.2.2 CHARCOAL LEAK TEST DID NOT HAVE INSTRUCTIONS FOR RETURNING THE EQUIPMENT TO ITS NORMAL OPERATING STATUS.
(7926 INFRACTION)

TECHNICAL SPECIFICATION 6.8.1 REQUIRES WRITTEN PROCEDURES BE FOLLOWED. SECTION 6.5.1 AND 6.5.3.2 OF THE RADIATION CONTROL AND PROTECTION MANUAL REQUIRES PERSONNEL TO WEAR LAB COATS WHEN REQUIRED. ON 7/24/79 TWO AUXILIARY OPERATORS ENTERED A CONTAMINATION AREA WITHOUT LAB COATS AS REQUIRED BY LOCAL POSTING.
(7926 INFRACTION)

SURVEILLANCE PROCEDURES NOT ESTABLISHED. PROCEDURES HAVE NOT BEEN ESTABLISHED FOR LICENSEE'S ENGINEERING AND QA/QC SURVEILLANCE ACTIVITIES RELATIVE TO INSERVICE INSPECTION. SURVEILLANCE PROCEDURES REQUIRED BY TECH SPECS.
(8023 INFRACTION)

INSERVICE INSPECTION PROGRAM INCOMPLETE. THE INSERVICE INSPECTION PROGRAM SUBMITTED TO NRC FOR REVIEW DOES NOT MEET REQUIREMENTS OF ASME B&PV CODE SECTION XI - NONCOMPLIANCE WITH 10CFR50.55 A(G).
(8023 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

- UNIT DOWN FOR REFUELING.
- LAST IE SITE INSPECTION DATE: MAY 3-31, 1980
- INSPECTION REPORT NO: 50-325/80-24

SUMMARY

INSPECTION SUMMARY

INSPECTION MAY 12-16 (80-19): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 35 INSPECTOR-HOURS ON SITE IN THE AREAS OF MAINTENANCE, SURVEILLANCE AND CALIBRATION OF TECHNICAL SPECIFICATION RELATED EQUIPMENT AND SUPPORT EQUIPMENTS. OF THE THREE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION MAY 27-30 (80-23): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 27 INSPECTOR-HOURS ON SITE IN THE AREAS OF INSERVICE INSPECTION (ISI) PROGRAM; MAINTENANCE AND CONTRACTOR WELDER TRAINING AND CERTIFICATION; IEB 80-07 JET PUMP BRACKET INSPECTION RESULTS (UNIT 2); SPECIAL RECIRCULATION NOZZLE INSPECTION PREPARATIONS (UNIT 1). LER 80-24 TORUS COATING BLISTERS (UNIT 2). OF THE FIVE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN FOUR AREAS; TWO ITEMS OF NONCOMPLIANCE WERE FOUND IN ONE AREA (INFRACTION - INSERVICE INSPECTION PROGRAM INCOMPLETE; AND INFRACTION - SURVEILLANCE ACTIVITY PROCEDURES NOT ESTABLISHED OR IMPLEMENTED).

INSPECTION MAY 3-31 (80-24): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 35 INSPECTOR-HOURS ON SITE IN THE AREAS OF CONTROL ROOM OBSERVATIONS, REVIEW OF OPERATOR LOGS, LIMITING CONDITIONS FOR OPERATION, TOUR OF VITAL AREAS, PHYSICAL SECURITY, RADIATION PROTECTION, HOUSEKEEPING AND CLEANLINESS, FIRE PROTECTION, REVIEW OF ACTION TAKEN ON IE BULLETINS AND REVIEW AND FOLLOWUP ON LICENSEE EVENT REPORTS. THIS ROUTINE INSPECTION BY THE RESIDENT INSPECTOR INVOLVED 35 MANHOURS OF INSPECTION TIME ON SITE IN THE TEN AREAS IDENTIFIED ABOVE. OF THE TEN AREAS INSPECTED NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

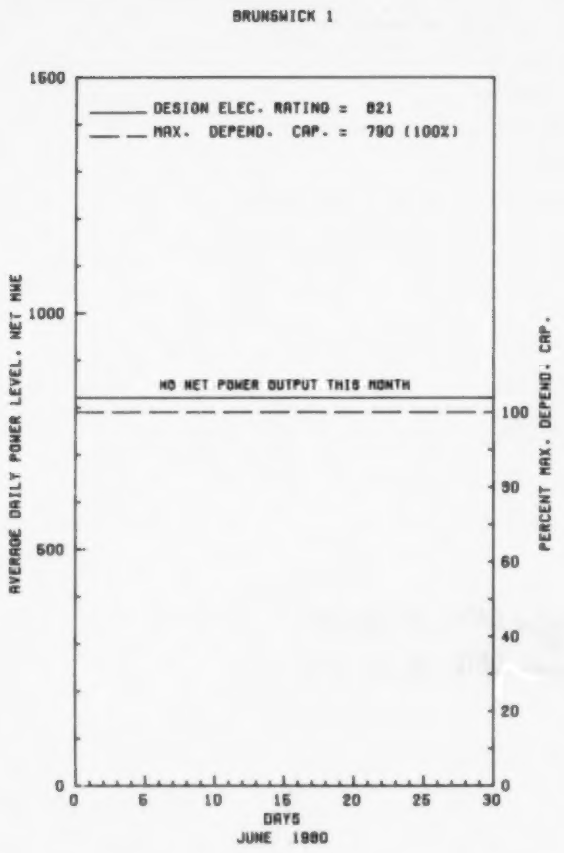
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-028/03L-0	04/14/80	05/08/80	CONTAINMENT ATMOSPHERE MONITOR FAILED DOWNSCALE.
80-038/03L-0	05/01/80	05/28/80	RECIRCULATION PUMP TRIPPED ON LOW OIL PRESSURE DUE TO ERRATIC OPERATION OF PRESSURE CONTROL VALVE.
80-039/03L-0	04/17/80	05/15/80	CONTAINMENT ATMOSPHERE ANALYZER 1263 FAILED.
80-040/03L-0	04/17/80	05/08/80	NUMBER 2 DG SERVICE WATER SUPPLY VALVES FAILED TO OPEN WHEN DG WAS STARTED.
80-041/03L-0	04/17/80	05/06/80	PARTIAL LOSS OF ROD POSITION FOR CONTROL ROD NO. 14-11.
80-043/03L-0	04/22/80	05/21/80	CONTROL POWER LOST TO NO. 4 DG.
80-044/03L-0	04/22/80	05/21/80	CONTAINMENT ATMOSPHERE CONTROL MONITOR RIP VALVE X57B DID NOT CLOSE FULLY.
80-045/03L-0	04/24/80	05/19/80	REACTOR LOW WATER LEVEL SWITCHES 2 AND 3 OUT OF CALIBRATION.
80-047/03L-0	05/01/80	05/28/80	RHR FLOW INDICATOR OUT OF CALIBRATION ON REMOTE SHUTDOWN PANEL.
80-048/03L-0	05/08/80	06/05/80	LEAKING GASKET ON CAL. ADJUSTMENT RX LOW LEVEL INST. CAUSED SWITCH TO FAIL.
80-049/03L-0	05/16/80	06/03/80	CONTROL ROD 14-15 DID NOT INDICATE AT POSITION 08.
80-051/03L-0	05/15/80	06/13/80	CONTAINMENT ATMOSPHERE DILUTION TANK VOLUME BELOW THE TECHNICAL SPECIFICATION LIMIT - SECOND OCCURRENCE.
80-052/03L-0	05/14/80	06/11/80	FAILURE OF A PRIMARY CONTAINMENT OXYGEN ANALYZER.

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 8-4-80

REVIEWED BY NRR DL: John N Hannon **DATE:** 8-6-80

BRUNSWICK 1

AVERAGE POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-325
2. Utility Contact: E. Willis (919) 457-9521
3. Licensed Thermal Power (MWt): 2436
4. Nameplate Rating (Gross MWe): 963 x 0.9 = 867
5. Design Electrical Rating (Net MWe): 821
6. Maximum Dependable Capacity (Gross MWe): 815
7. Maximum Dependable Capacity (Net MWe): 790
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	4,367.0	28,824.0
12. Hours Reactor Was Critical	.0	3,270.4	20,660.0
13. Reactor Reserve Shutdown Hours	.0	.0	1,647.1
14. Hours Generator On-Line	.0	3,148.3	19,481.1
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	6,237,934	40,067,430
17. Gross Electrical Energy (MWH)	0	2,087,474	13,326,293
18. Net Electrical Energy (MWH)	0	2,011,410	12,819,220
19. Unit Service Factor	.0	72.1	67.6
20. Unit Availability Factor	.0	72.1	67.6
21. Unit Capacity Factor (MDC Net)	.0	58.3	56.3
22. Unit Capacity Factor (DER Net)	.0	56.1	54.2
23. Unit Forced Outage Rate	.0	10.5	19.9

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): MAINTENANCE OUTAGE, 09/19/80.
25. If Shut Down At End of Report Period, Estimated Date of Startup: 07/08/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	05/26/80	S	720.0	C	4		RC	FUELXX	REFUELING OUTAGE.

SUMMARY

THE UNIT WAS SHUTDOWN FOR REFUELING AND MAINTENANCE OUTAGE.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

NRC POWER RESTRICTIONS: NONE

BRUNSWICK 2

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	North Carolina/Brunswick 3 miles N of Southport
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2436
5.	Design Electrical Rating (MWE Net)	821
6.	Date of Initial Criticality	March 20, 1975
7.	Date of Commercial Operation	November 3, 1975
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Cape Fear River

Utility & Contractor information

10.	Licensee	Carolina Power and Light Company
11.	Corporate Address	411 Fayetteville Street Raleigh, North Carolina 27602
12.	Corporate Contact	J. A. Jones Executive Vice President
13.	Architect/Engineer	United Engineers & Constructors, Inc.
14.	Nuclear Steam Supply System	GE
15.	Constructor	Brown and Root, Inc.

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R.C. Lewis
18.	IE Section Chief	P.J. Kellogg
19.	IE Resident Inspector	J. E. Outzs (919) 457-9531
20.	Licensing Project Manager	J. Hannon
21.	Docket Number	50-324
22.	License Number/Date of Issue	DPR-62, 12/27/74
23.	Public Document Room	Southport-Brunswick County Library 109 W. Moore Street Southport, North Carolina 28461

INSPECTION STATUS

ENFORCEMENT STATUS

AS REQUIRED BY SECTION 3.7.2.2 OF THE TECHNICAL SPECIFICATIONS, WITH ONE OR MORE OF THE DIESEL GENERATOR SPRAY AND/OR SPRINKLER SYSTEMS INOPERABLE, ESTABLISH A CONTINUOUS FIRE WATCH WITH BACKUP FIRE SUPPRESSION EQUIPMENT WITHIN ONE HOUR. CONTRARY TO THE ABOVE, ON OCTOBER 16 THE INSPECTOR FOUND FIRE PROTECTION SPRINKLER AND STANDPIPE SYSTEMS ISOLATED TO DIESEL GENERATOR NO. 3. THE ISOLATED SYSTEMS WERE INDICATED BY LOW PRESSURE ANNUNCIATORS FOR THE SYSTEMS. A FOLLOWUP INVESTIGATION REVEALED THAT TWO DELUGE VALVES (DV-15 AND DV-17) HAD BEEN ISOLATED FOR MAINTENANCE BY MANUALLY CLOSING THREE ISOLATION VALVES. THIS ACTION WAS TAKEN WITHOUT NOTIFYING OPERATING PERSONNEL, AND HAVING THE SYSTEM PLACED IN A LIMITING CONDITION FOR OPERATION (LCO) AND TAKING COMPENSATORY MEASURES REQUIRED BY THE TECHNICAL SPECIFICATIONS. AN LCO WAS ESTABLISHED AFTER THE INSPECTOR FOUND THE CONDITION, NOTIFIED OPERATING PERSONNEL AND THEY HAD INVESTIGATED THE CAUSE OF THE ANNUNCIATOR ALARM.
(7937 INFRACTION)

SURVEILLANCE PROCEDURES NOT ESTABLISHED. PROCEDURES HAVE NOT BEEN ESTABLISHED FOR LICENSEE'S ENGINEERING AND QA/QC SURVEILLANCE ACTIVITIES RELATIVE TO INSERVICE INSPECTION. SURVEILLANCE PROCEDURES REQUIRED BY TECH SPECS.
(8020 INFRACTION)

INSERVICE INSPECTION PROGRAM INCOMPLETE. THE INSERVICE INSPECTION PROGRAM SUBMITTED TO NRC FOR REVIEW DOES NOT MEET REQUIREMENTS OF ASME B&PV CODE SECTION XI - NONCOMPLIANCE WITH 10CFR50.55 A(G).
(8020 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

UNIT SHUTDOWN FOR REFUELING AND TORUS MODIFICATION.

LAST IE SITE INSPECTION DATE: MAY 3-31, 1980

INSPECTION REPORT NO: 50-324/80-21

SUMMARY

INSPECTION SUMMARY

INSPECTION MAY 12-16 (80-16): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 35 INSPECTOR-HOURS ON SITE IN THE AREAS OF MAINTENANCE, SURVEILLANCE AND CALIBRATION OF TECHNICAL SPECIFICATION RELATED EQUIPMENT AND SUPPORT EQUIPMENTS. OF THE THREE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION MAY 27-30 (80-20): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 27 INSPECTOR-HOURS ON SITE IN THE AREAS OF INSERVICE INSPECTION (ISI) PROGRAM; MAINTENANCE AND CONTRACTOR WELDER TRAINING AND CERTIFICATION; IEB 80-07 JET PUMP BRACKET INSPECTION RESULTS (UNIT 2); SPECIAL RECIRCULATION NOZZLE INSPECTION PREPARATIONS (UNIT 1), LER 80-24 TORUS COATING BLISTERS (UNIT 2). OF THE FIVE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN FOUR AREAS; TWO ITEMS OF NONCOMPLIANCE WERE FOUND IN ONE AREA (INFRACTION - INSERVICE INSPECTION PROGRAM INCOMPLETE AND INFRACTION - SURVEILLANCE ACTIVITY PROCEDURES NOT ESTABLISHED OR IMPLEMENTED).

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-021/03L-0	05/16/80	06/13/80	DRIFT OF REACTOR PRESSURE LOW SETPOINTS EXCEEDS SPECIFICATION.
80-030/03L-0	04/12/80	05/08/80	PARTIALLY BUCKLED RIB PLATE ON 2B RHR HEAT EXCHANGER.
80-036/03L-0	04/18/80	05/14/80	MAIN TURBINE CONTROL VALVE FAST CLOSURE TEST FAILED.
80-037/03L-0	05/02/80	05/30/80	IRM CHANNEL B WAS DAMAGED WHILE REMOVING A CRD UNIT FOR MAINTENANCE.
80-040/03L-0	04/24/80	05/23/80	FAILURE OF INTERMEDIATE RANGE MONITORS.
80-041/03L-0	04/25/80	05/19/80	REACTOR LEVEL SWITCH OUT OF CALIBRATION.
80-043/03L-0	05/07/80	06/03/80	FAILURE TO PERFORM SURVEILLANCE TEST ON STANDBY GAS TREATMENT SYSTEM.
80-044/03L-0	05/16/80	06/11/80	FAILURE OF SOURCE RANGE MONITORS.
80-046/01T-0	06/11/80	06/12/80	NON-ENVIRONMENTALLY QUALIFIED SPLICES FOUND ON THE S/R VALVE SOLENOID MAKEUP.

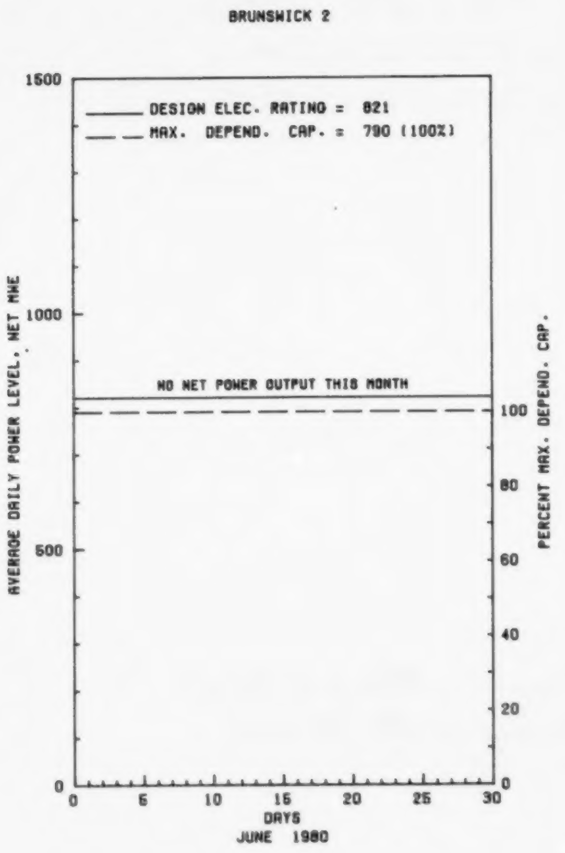
INSPECTION MAY 3-31 (80-21): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 35 INSPECTOR-HOURS ON SITE IN THE AREAS OF CONTROL ROOM OBSERVATIONS, REVIEW OF OPERATOR LOGS, LIMITING CONDITIONS FOR OPERATION, TOUR OF VITAL AREAS, PHYSICAL SECURITY, RADIATION PROTECTION, HOUSEKEEPING AND CLEANLINESS, FIRE PROTECTION, REVIEW OF ACTION TAKEN ON IE BULLETINS AND REVIEW AND FOLLOWUP ON LICENSEE EVENT REPORTS. THIS ROUTINE INSPECTION BY THE RESIDENT INSPECTOR INVOLVED 35 MANHOURS OF INSPECTION TIME ON SITE IN THE TEN AREAS IDENTIFIED ABOVE. OF THE TEN AREAS INSPECTED NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

REVIEWED BY OMPA LOEB: John J Riggs **DATE:** 8-4-80

REVIEWED BY NRR DL: John N. Hannon **DATE:** 8-6-80

BRUNSWICK 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-324
2. Utility Contact: E. Willis (919) 457-9521
3. Licensed Thermal Power (MWt): 2436
4. Nameplate Rating (Gross MWe): 963 x 0.9 = 867
5. Design Electrical Rating (Net MWe): 821
6. Maximum Dependable Capacity (Gross MWe): 815
7. Maximum Dependable Capacity (Net MWe): 790
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE.

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	4,367.0	40,845.0
12. Hours Reactor Was Critical	.0	1,365.7	26,689.6
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	.0	1,228.1	25,192.1
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	2,694,960	46,339,422
17. Gross Electrical Energy (MWH)	0	872,613	15,487,687
18. Net Electrical Energy (MWH)	0	842,126	14,882,598
19. Unit Service Factor	.0	28.1	61.5
20. Unit Availability Factor	.0	28.1	61.5
21. Unit Capacity Factor (MDC Net)	.0	24.4	46.1
22. Unit Capacity Factor (DER Net)	.0	23.5	44.4
23. Unit Forced Outage Rate	.0	8.1	12.8

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 07/06/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	03/01/80	S	720.0	C	4		RC	FUELXX	REFUELING CONTINUES.

SUMMARY

THE UNIT WAS SHUTDOWN FOR REFUELING AND MAINTENANCE OUTAGE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

CALVERT CLIFFS 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Maryland/Calvert 40 miles S of Annapolis
2.	Electric Reliability Council	Mid-Atlantic Area Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2700
5.	Design Electrical Rating (MWE Net)	845
6.	Date of Initial Criticality	October 7, 1974
7.	Date of Commercial Operation	May 8, 1975
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Chesapeake Bay

Utility & Contractor Information

10.	Licensee	Baltimore Gas and Electric Company
11.	Corporate Address	P. O. Box 1475 Baltimore, Maryland 21203
12.	Corporate Contact	A. E. Lundvall, Jr. Vice President-Supply
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Combustion Engineering
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Section Chief	E. C. McCabe, Jr.
19.	IE Resident Inspector	R. Architzel (301) 586-2626
20.	Licensing Project Manager	E. L. Conner, Jr.
21.	Docket Number	50-317
22.	License Number/Date of Issue	DPR-53, 7/31/74
23.	Public Document Room	Calvert County Library Prince Frederick, Maryland 20678

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS:

No significant new information.

- LAST IE SITE INSPECTION DATE: 6/30/80
- INSPECTION REPORT NO: 50-317/80-10

SUMMARY

INSPECTION SUMMARY

- 50-317/80-05 - Apr 7-30: Routine, onsite regular and back-shift inspection by the resident inspector (20 hours). Areas inspected included the control room and the accessible portions of the auxiliary, turbine, service, and intake buildings; radiation protection; physical security; fire protection; plant operating records; spent fuel shipment, loss of coolant accident procedure and associated training review, and reporting to the NRC. No items of noncompliance were identified.
- 50-317/80-07 - Apr 29 - May 1: Unannounced investigation by one regional based investigator (16 hours) of allegations relating to the pipe support activities performed to comply with NRC Bulletin 79-02 and radiation safety procedures. Portions of the allegations were investigated through several telephone contacts with licensee and contractor representatives. Findings in these areas were substantiated by documentation obtained at the site by the Senior Resident Inspector. None of the allegations were substantiated. No items of noncompliance were identified.

REPORTS RECEIVED FROM LICENSEE

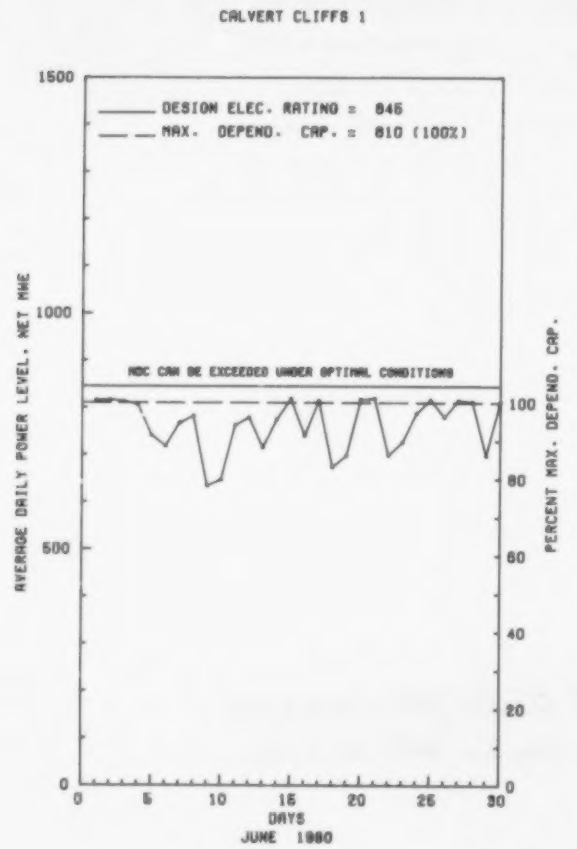
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-024/03L	04/30/80	05/30/80	Cracked weld discovered on 11B RCP controlled bleed off line apparent cause was vibration fatigue, thermal cycles and/or brittleness during welding eng eval to be performed.
80-026/01T	05/20/80	06/03/80	Aux feed decay heat removal shifted to main feed after trip due service water loss and later restoring of service water, aux feed suctions were isolated due operator error.
80-027/01T	05/20/80	06/03/80	Manual trip when both service water systems common suction header air-bound, necessitating decay heat removal by aux feed until service water supply restored to main feed
80-028/03L	05/15/80	06/13/80	Diesel generator No. 12 failed to reach rated speed within 10 seconds. Investigation revealed intermittent female contact pin on amphenol connector, 250 rpm low speed switch.

REVIEWED BY OMPA LOEB: T. C. [illegible] **DATE:** 8.4.80

REVIEWED BY NRR DL: Cheng-Yang Li **DATE:** 8/15/80

CALVERT CLIFFS 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-317
2. Utility Contact: S. D. Merson (301) 234-5240
3. Licensed Thermal Power (MWt): 2700
4. Nameplate Rating (Gross MWe): 1020 x 0.9 = 918
5. Design Electrical Rating (Net MWe): 845
6. Maximum Dependable Capacity (Gross MWe): 845
7. Maximum Dependable Capacity (Net MWe): 810
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	4,367.0	45,132.0
12. Hours Reactor Was Critical	720.0	3,803.0	36,242.6
13. Reactor Reserve Shutdown Hours	.0	38.3	1,098.8
14. Hours Generator On-Line	720.0	3,733.6	35,435.6
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,833,401	8,629,180	83,966,168
17. Gross Electrical Energy (MWH)	575,757	2,724,149	27,536,679
18. Net Electrical Energy (MWH)	549,545	2,583,776	26,232,982
19. Unit Service Factor	100.0	85.5	78.5
20. Unit Availability Factor	100.0	85.5	78.5
21. Unit Capacity Factor (MDC Net)	94.2	73.0	71.8
22. Unit Capacity Factor (DER Net)	90.3	70.0	68.8
23. Unit Forced Outage Rate	.0	4.9	8.6

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
PLANNED OUT. 10/17/80 FOR REFUEL, UNIT GEN. INSP. & TMI MOD.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
NONE									

SUMMARY

THE UNIT RAN AT OR NEAR FULL POWER THE ENTIRE MONTH.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

CALVERT CLIFFS 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Maryland/Calvert 40 miles S of Annapolis
2.	Electric Reliability Council	Mid-Atlantic Area Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2700
5.	Design Electrical Rating (MWE Net)	845
6.	Date of Initial Criticality	November 30, 1976
7.	Date of Commercial Operation	April 1, 1977
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Chesapeake Bay

Utility & Contractor Information

10.	Licensee	Baltimore Gas and Electric Company
11.	Corporate Address	P. O. Box 1475 Baltimore, Maryland 21203
12.	Corporate Contact	A. E. Lundvall, Jr. Vice President-Supply
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Combustion Engineering
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	I E Section Chief	E. C. MCCABE, JR.
19.	IE Resident Inspector	R. Architzel (301) 586-2626
20.	Licensing Project Manager	E. L. Conner, Jr.
21.	Docket Number	50-318
22.	License Number/Date of Issue	DPR-69, 8/13/76
23.	Public Document Room	Calvert County Library Prince Frederick, Maryland 20678

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO TS 6.8.1 D AND REGULATORY GUIDE 1.33, OI-17C WAS NOT APPROPRIATE. IN THAT PERFORMANCE OF THE STEPS IN ORDER WOULD HAVE RESULTED IN OPENING DEBORATION ION EXCHANGER 21 ISOLATION BY OPENING THE VENT BEFORE SHUTTING THE LETDOWN STRAINER ISOLATION VALVES. CAUSING AN UNPLANNED RELEASE OF RADIOACTIVITY
(8002 INFRACTION)

CONTRARY TO TS 6.8.1.A AND OI-17C, ON FEBRUARY 11, 1980, REQUIRED 2 VALVE ISOLATION OF DEBORATING ION EXCHANGER 21 WAS NOT ACCOMPLAINED AND THE SINGLE, UNSPECIFIED VALVE USED FOR ISOLATION WAS NOT SHUT BECAUSE OF ENTANGLED WIRE (DUE TO FAULTY HOUSEKEEPING, COMBINED WITH DEFECTIVE POSITION INDICATION, CAUSING A RADIATION RELEASE TO THE ENVIRONS.
(8002 INFRACTION)

CONTRARY TO TS 6.8.1.A AND OI-17C, ON FEBRUARY 12, 1980, WHEN DEBORATING ION EXCHANGER 21 WAS BEING RESTORED TO SERVICE. ITS VENT WAS NOT FULLY SHUT BEFORE OPENING OF VALVES EFFECTING ISOLATION RESULTED IN AN UNPLANNED RELEASE OF RADIOACTIVITY.
(8002 INFRACTION)

CONTRARY TO TS 6.8.1.E AND SEPIP A, AN UNPLANNED RELEASE ON FEBRUARY 11, 1980 RESULTED IN A PLANT EMERGENCY BEING DECLARED WITHOUT FOLLOWUP NOTIFICATION BEING MADE TO THE NRC.
(8002 INFRACTION)

CONTRARY TO TS 6.8.1.E AND SEPIP A, NO ESTIMATE OF OFFSITE DOSE WAS MADE ON FEBRUARY 11, 1980 WHEN A PLANT EMERGENCY INVOLVING A RELEASE TO THE ENVIRONS OCCURRED.
(8002 INFRACTION)

CONTRARY TO TS 6.8.1.E AND SEPIP A, RE-ENTRY INTO THE EVACUATED AUXILIARY BUILDING WAS MADE WITHOUT THE REQUIRED BRIEFING ON THE HAZARDS INVOLVED.
(8002 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS:

No significant new information.

* LAST IE SITE INSPECTION DATE: 6/30/80

* INSPECTION REPORT NO: 50-318/80-09

SUMMARY

INSPECTION SUMMARY

* 50-318/80-05 - Apr 7-30: Routine, onsite regular and backshift inspection by the resident inspector (20 hours). Areas inspected included the control room and the accessible portions of the auxiliary, turbine, service, and intake buildings; radiation protection; physical security; fire protection; plant operating records; spent fuel shipment, loss of coolant accident procedure and associated training review, and reporting to the NRC. No items of noncompliance were identified.

REPORTS RECEIVED FROM LICENSEE

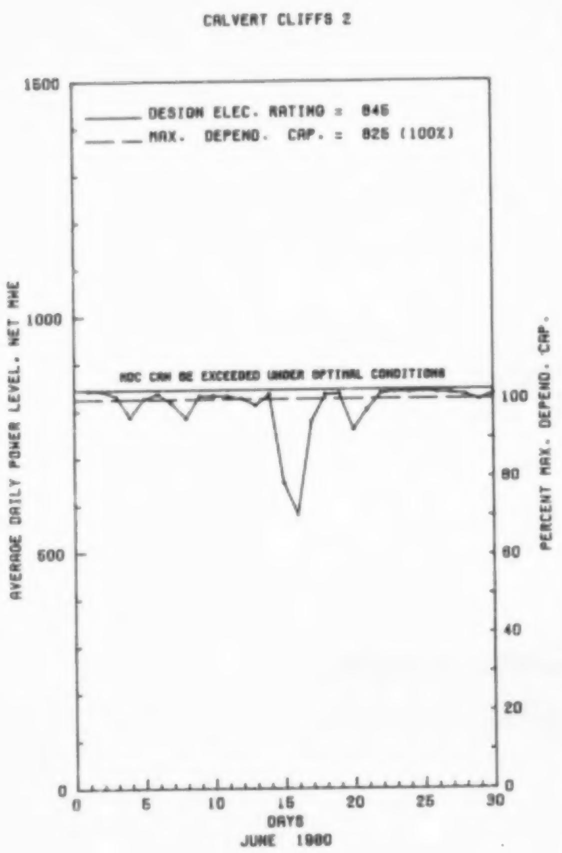
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-027/03L	05/06/80	06/02/80	Channel B RPS trip units bypassed to investigate channel B axial flux offset indication caused by dirty connector on the power summer and subchannel deviation card.
80-028/03L	05/12/80	06/02/80	Plant computer inoperable for 40 minutes. Shutdown caused by noise spike when connecting test equipment.
80-029/03L	05/15/80	06/13/80	RCS hot leg sample isolation valve would not indicate closed. Valve position could not be verified, redundant containment isolation valve closed.
80-030/03L	05/17/80	06/13/80	Containment particulate and gaseous monitors OOS due to a grounded condition on the instrument bus 2Y10.
80-031/03L	05/22/80	06/13/80	Containment gaseous and particulate monitors out of service due to technician error during circuit restoration

REVIEWED BY OMPA LOEB: MR Beebe **DATE:** 08/04/80

REVIEWED BY NRR DL: Chung-Yang Li **DATE:** 8/15/80

CALVERT CLIFFS 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-318
2. Utility Contact: S. D. Merson (301) 234-5240
3. Licensed Thermal Power (MWt): 2700
4. Nameplate Rating (Gross MWe): 1012 x 0.9 = 911
5. Design Electrical Rating (Net MWe): 845
6. Maximum Dependable Capacity (Gross MWe): 860
7. Maximum Dependable Capacity (Net MWe): 825
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

ITEM 21 CALCULATED WITH A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	4,367.0	28,487.0
12. Hours Reactor Was Critical	720.0	4,127.8	24,872.7
13. Reactor Reserve Shutdown Hours	.0	9.5	422.6
14. Hours Generator On-Line	720.0	4,102.0	23,710.5
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,869,379	10,658,482	58,922,713
17. Gross Electrical Energy (MWH)	609,343	3,559,297	19,564,520
18. Net Electrical Energy (MWH)	583,087	3,405,576	18,662,596
19. Unit Service Factor	100.0	93.9	83.2
20. Unit Availability Factor	100.0	93.9	83.2
*21. Unit Capacity Factor (MDC Net)	98.2	94.5	80.3
22. Unit Capacity Factor (DER Net)	95.8	92.3	77.5
23. Unit Forced Outage Rate	.0	.9	5.9

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-03	06/15/80	F	48.0	A	5			PUMPXX	VARIOUS FORCED REDUCTIONS TO REPAIR NO. 22 STEAM GENERATOR FEED PUMP.

SUMMARY

THE PLANT AT FULL POWER EXCEPT FOR REDUCTIONS FOR STEAM GENERATOR FEED PUMP REPAIR.

LEGEND

[1] Type:
F-Forced
S-Scheduled

[2] Reason:
A-Equipment Failure
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error
H-Other

[3] Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Continuations
5-Load Reduction
9-Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

COOK 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Michigan/Berrien 11 miles S of Benton Harbor
2.	Electric Reliability Council	East Central Area Reliability Coordination Agreement
3.	Type of Reactor	PWR
4.	Power Level (MWT)	3250
5.	Design Electrical Rating (MWE Net)	1054
6.	Date of Initial Criticality	January 18, 1975
7.	Date of Commercial Operation	August 27, 1975
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10.	Licensee	AEP Corporation
11.	Corporate Address	Indiana & Michigan Power Co. 2 Broadway New York, New York 10004
12.	Corporate Contact	John Dolan Senior Executive V. P. Eng.
13.	Architect/Engineer	AEP
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Indiana & Michigan Power Co.

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	D.C. Boyd
19.	IE Resident Inspector	R. Masse (616) 465-6775
20.	Licensing Project Manager	S. Miner
21.	Docket Number	50-315
22.	License Number/Date of Issue	DPR-58 issued 10/25/74
23.	Public Document Room	Maude Preston Palenske Memorial Library 500 Market Street St. Joseph, Michigan 49085

INSPECTION STATUS

ENFORCEMENT STATUS

10 CFR 50, APPENDIX B, CRITERION II ESTABLISHED THE REQUIREMENT FOR ADEQUATE CLEANLINESS CONTROL OVER ACTIVITIES AFFECTING THE QUALITY OF IDENTIFIED STRUCTURES, SYSTEMS AND COMPONENTS. PMI 2090 REQUIRES PERIODIC CLEANLINESS INSPECTIONS OF THE PLANT. CONTRARY TO THE ABOVE, THE IN-PLANT GENERAL SAFETY COMMITTEE HAD NOT CONDUCTED QUARTERLY CLEANLINESS INSPECTIONS AS REQUIRED.
(8001 DEFICIENCY)

10 CFR 50, APPENDIX B, CRITERION II STATES THAT THE APPLICANT SHALL IDENTIFY THE STRUCTURES SYSTEMS, AND COMPONENTS TO BE COVERED BY THE QUALITY ASSURANCE PROGRAM. FSAR SECTION 1.7.19.2 STATES THAT AN "N" LIST OF CLASS I COMPONENTS HAS BEEN DEVELOPED AND IS MAINTAINED BY AEPSC. CONTRARY TO THE ABOVE, THE INSPECTOR FOUND THAT THE "N" LIST HAD NOT BEEN MAINTAINED AS REQUIRED. THE "N" LIST CURRENTLY IN USE IS IN DRAFT FORM AND HAD NOT BEEN APPROVED.
(8001 DEFICIENCY)

10 CFR 50, APPENDIX B, CRITERION XI REQUIRES THAT A TEST PROGRAM SHALL BE ESTABLISHED TO ASSURE THAT ALL TESTING REQUIREMENTS TO DEMONSTRATE THAT STRUCTURES, SYSTEMS AND COMPONENTS WILL PERFORM SATISFACTORILY IN SERVICE. PMI 4030 REQUIRES THAT WHEN CHANGES TO THE TECHNICAL SPECIFICATIONS ARE MADE, THE MASTER SURVEILLANCE TEST TABLE SHALL BE REVIEWED AND REVISED AS NECESSARY. CONTRARY TO THE ABOVE, THE INSPECTOR FOUND THAT AN UP-TO-DATE MASTER SURVEILLANCE SCHEDULE WAS NOT BEING MAINTAINED.
(8001 DEFICIENCY)

10 CFR 50, APPENDIX B, CRITERION XIV REQUIRES THAT PROCEDURES SHALL BE PROVIDED FOR CONTROL OF EQUIPMENT, AS NECESSARY, TO MAINTAIN PERSONNEL AND REACTOR SAFETY AND TO AVOID UNAUTHORIZED OPERATION OF EQUIPMENT. PMI 2140 REQUIRES THAT LIFTED LEADS SHALL BE TAGGED TO IDENTIFY THEM FOR RETERMINATION. CONTRARY TO THE ABOVE, THE INSPECTOR FOUND THAT LEADS OF THE INSTRUMENT AND CONTROL CABLES FOR THE HOT SHUTDOWN PANEL WERE NOT BEING CONTROLLED AS REQUIRED.
(8001 INFRACTION)

10 CFR 50, APPENDIX B, CRITERION XVIII STATES THAT A COMPREHENSIVE SYSTEM OF PLANNED AND PERIODIC AUDITS SHALL BE CARRIED OUT TO VERIFY COMPLIANCE WITH ALL ASPECTS OF THE QUALITY ASSURANCE PROGRAM AND TO DETERMINE THE EFFECTIVENESS OF THE PROGRAM. CONTRARY TO THE ABOVE, THE INSPECTOR FOUND THAT A COMPREHENSIVE SYSTEM OF AUDITS FOR SAFETY RELATED ACTIVITIES AT THE CORPORATE OFFICE HAD NOT BEEN ADDRESSED IN THE AEPSC QUALITY ASSURANCE PROGRAM AND THAT FORMAL AUDITS OF CORPORATE OFFICE ACTIVITIES HAD NOT BEEN IMPLEMENTED.
(8001 INFRACTION)

10 CFR 50, APPENDIX B, CRITERION V STATES THAT ACTIVITIES EFFECTING QUALITY SHALL BE PRESCRIBED BY DOCUMENTED INSTRUCTIONS AND PROCEDURES AND SHALL BE ACCOMPLISHED IN ACCORDANCE WITH THESE INSTRUCTIONS AND PROCEDURES. PMI 7020 REQUIRES THAT IF CORRECTIVE ACTION IS REQUIRED AS PART OF AN AUDIT REPORT, ACTION SHALL BE TAKEN WITHIN 30 DAYS. CONTRARY TO THE ABOVE, CORRECTIVE ACTION FOR AUDIT REPORTS QA 78-12, 78-19 AND 78-21 WAS NOT TAKEN IN THE REQUIRED 30 DAY PERIOD.
(8001 INFRACTION)

TECHNICAL SPECIFICATION 6.8.1 REQUIRES IMPLEMENTATION OF PROCEDURES. 12-OHP-4021.018.004 "DRAINING OR ADJUSTING LEVEL IN THE SPENT FUEL PIT" DETAILS THE PROCEDURE FOR RAISING LEVEL IN THE SFP. CONTRARY TO THE ABOVE, THIS PROCEDURE WAS NOT USED FOR INCREASING LEVEL IN THE SFP BETWEEN MARCH 15, 1980 AND MARCH 17, 1980, WHEN THE POOL WAS OVERFILLED AND CROSS-CONTAMINATION OF THE DEMINERALIZED WATER SYSTEM OCCURRED.
(8004 INFRACTION)

AMENDMENT NUMBER 32 TO NRC OPERATING LICENSE NUMBER DPR-58 AND AMENDMENT NUMBER 13 TO DPR-74 REQUIRE A MINIMUM OF 2000 PPM BORON CONCENTRATION IN THE SPENT FUEL PIT DURING CHANGEOUT FROM OLD RACKS TO NEW HIGH DENSITY FUEL STORAGE RACKS. CONTRARY TO THE ABOVE, ON MARCH 17, 1980, THE SFP CONCENTRATION WAS FOUND TO BE LESS THAN 2000 PPM (1954 PPM).
(8004 INFRACTION)

AS REQUIRED BY 10 CFR 71.5, SHIPMENTS OF LICENSED RADIOACTIVE MATERIAL MUST COMPLY WITH THE REQUIREMENTS OF THE DEPARTMENT OF TRANSPORTATION, 49 CFR 170-189, WHICH REQUIRES PLACARDING OF EACH END AND SIDE OF EXCLUSIVE USE VEHICLES TRANSPORTING LSA MATERIAL. (PARAGRAPH 49 CFR 172.504, 172.506, AND 173.392(C)(7)). CONTRARY TO THE ABOVE, ON FEBRUARY 29, 1980, THREE EXCLUSIVE USE TRUCKS CARRYING LARGE BOXES OF LSA MATERIAL FROM THE D. C. COOK SITE TO THE NUCLEAR ENGINEERING COMPANY, BEATTY, NEVADA, WERE FOUND UPON ARRIVAL TO BE MISSING THE FRONT PLACARD.
(8009 DEFICIENCY)

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

FACILITY ITEMS

- Unit 1 shutdown on May 30, 1980 to begin a 61 day maintenance and refueling outage.

PLANT STATUS

- Reactor shutdown at 9:00 p.m. on May 30, 1980, to begin a scheduled 61 day refueling outage.

*LAST IE SITE INSPECTION DATE: February 29, 1980

- INSPECTION REPORT NO: 80-09

SUMMARY

INSPECTION SUMMARY

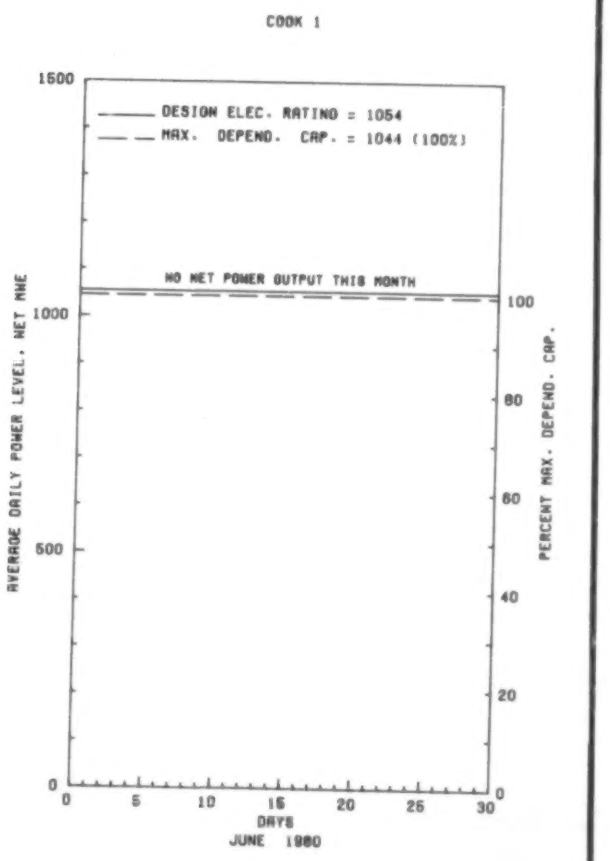
- Meeting on April 15, (80-07): This special management meeting was held to discuss NRC concerns in two areas: implementation of the American Electric Company Corporate Quality Assurance Program; and the D. C. Cook Plant regulatory performance in the areas of personnel error and adherence to procedure. No items of noncompliance were identified as a result of this meeting.
- Inspection on February 29, (80-09): As a part of the inspection conducted during February 25 through March 7, 1980, at Beatty, Nevada, this report details the inspection of trailers No. 232214, No. 331143, and No. 331011, all from D. C. Cook. The shipments consisted of large wooden boxes on an open flatbed trailer. Three apparent items of noncompliance (incomplete placarding) with DOT/NRC regulations were observed.

REVIEWED BY OMPA LOEB: Philip A. Ross **DATE:** 08/04/80

REVIEWED BY NRR DL: Sydney Miner fo **DATE:** 8/5/80

COOK 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-315
2. Utility Contact: W. T. Gillett (616) 465-5901
3. Licensed Thermal Power (MWt): 3250
4. Nameplate Rating (Gross MWe): 1280 x 0.9 = 1152
5. Design Electrical Rating (Net MWe): 1054
6. Maximum Dependable Capacity (Gross MWe): 1080
7. Maximum Dependable Capacity (Net MWe): 1044
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

		MONTH	YEAR	CUMULATIVE
11.	Hours in Reporting Period	720.0	4,367.0	45,191.0
12.	Hours Reactor Was Critical	.0	3,287.5	36,163.6
13.	Reactor Reserve Shutdown Hours	.0	0	463.0
14.	Hours Generator On-Line	.0	3,199.7	35,282.9
15.	Unit Reserve Shutdown Hours	.0	.0	321.0
16.	Gross Thermal Energy (MWH)	0	10,118,931	99,185,236
17.	Gross Electrical Energy (MWH)	0	3,347,810	32,520,150
18.	Net Electrical Energy (MWH)	0	3,245,772	31,243,116
19.	Unit Service Factor	.0	73.3	76.7
20.	Unit Availability Factor	.0	73.3	76.7
21.	Unit Capacity Factor (MDC Net)	.0	71.2	67.4
22.	Unit Capacity Factor (DER Net)	.0	70.5	62.9
23.	Unit Forced Outage Rate	.0	11.6	7.3

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 08/04/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
160	05/30/80	S	720.0	C	4		RC	FUELXX	THE UNIT WAS REMOVED FROM SERVICE FOR CYCLE IV - V REFUELING OUTAGE AT 2228 HOURS EDT ON 05/30/80. THE UNIT REMAINED OUT OF SERVICE THE ENTIRE MONTH.

SUMMARY

THE UNIT WAS SHUTDOWN FOR REFUELING AND MAINTENANCE OUTAGE.

LEGEND

[1] Type:
- F–Forced
- S–Scheduled

[2] Reason:
- A–Equipment Failure
- B–Maintenance or Test
- C–Refueling
- D–Regulatory Restriction
- E–Operator Training & License Examination
- F–Administrative
- G–Operational Error
- H–Other

[3] Method:
- 1–Manual
- 2–Manual Scram
- 3–Automatic Scram
- 4–Continuations
- 5–Load Reduction
- 9–Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

COOK 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Michigan/Berrien 11 miles S of Benton Harbor
2.	Electric Reliability Council	East Central Area Reliability Coordination Agreement
3.	Type of Reactor	PWR
4.	Power Level (MWT)	3391
5.	Design Electrical Rating (MWE Net)	1100
6.	Date of Initial Criticality	March 10, 1978
7.	Date of Commercial Operation	July 1, 1978
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10.	Licensee	AEP Corporation Indiana & Michigan Power Co. Indiana & Michigan Electric Co.
11.	Corporate Address	2 Broadway New York, New York 10004
12.	Corporate Contact	John Dolan, VP/Chief Engineer (212) 422 - 4800
13.	Architect/Engineer	American Elec. Power Service Corp.
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Indiana & Michigan Power Co.

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	D.C. Boyd
19.	IE Resident Inspector	R. Masse (616) 465-6775
20.	Licensing Project Manager	S. Miner
21.	Docket Number	50-316
22.	License Number/Date of Issue	DPR-74, 12/23/77
23.	Public Document Room	Maude Preston Palenske Memorial Library 500 Market Street St. Joseph, Michigan 49085

INSPECTION STATUS

ENFORCEMENT STATUS

10 CFR 50, APPENDIX B, CRITERION II ESTABLISHED THE REQUIREMENT FOR ADEQUATE CLEANLINESS CONTROL OVER ACTIVITIES AFFECTING THE QUALITY OF IDENTIFIED STRUCTURES, SYSTEMS AND COMPONENTS. PMI 2090 REQUIRES PERIODIC CLEANLINESS INSPECTIONS OF THE PLANT. CONTRARY TO THE ABOVE, THE IN-PLANT GENERAL SAFETY COMMITTEE HAD NOT CONDUCTED QUARTERLY CLEANLINESS INSPECTIONS AS REQUIRED.
(8001 DEFICIENCY)

10 CFR 50, APPENDIX B, CRITERION II STATES THAT THE APPLICANT SHALL IDENTIFY THE STRUCTURES SYSTEMS, AND COMPONENTS TO BE COVERED BY THE QUALITY ASSURANCE PROGRAM. FSAR SECTION 1.7.19.2 STATES THAT AN "N" LIST OF CLASS I COMPONENTS HAS BEEN DEVELOPED AND IS MAINTAINED BY AEPSC. CONTRARY TO THE ABOVE, THE INSPECTOR FOUND THAT THE "N" LIST HAD NOT BEEN MAINTAINED AS REQUIRED. THE "N" LIST CURRENTLY IN USE IS IN DRAFT FORM AND HAD NOT BEEN APPROVED
(8001 DEFICIENCY)

10 CFR 50, APPENDIX B, CRITERION XI REQUIRES THAT A TEST PROGRAM SHALL BE ESTABLISHED TO ASSURE THAT ALL TESTING REQUIREMENTS TO DEMONSTRATE THAT STRUCTURES, SYSTEMS AND COMPONENTS WILL PERFORM SATISFACTORILY IN SERVICE. PMI 4030 REQUIRES THAT WHEN CHANGES TO THE TECHNICAL SPECIFICATIONS ARE MADE, THE MASTER SURVEILLANCE TEST TABLE SHALL BE REVIEWED AND REVISED AS NECESSARY. CONTRARY TO THE ABOVE, THE INSPECTOR FOUND THAT AN UP-TO-DATE MASTER SURVEILLANCE SCHEDULE WAS NOT BEING MAINTAINED.
(8001 DEFICIENCY)

10 CFR 50, APPENDIX B, CRITERION XIV REQUIRES THAT PROCEDURES SHALL BE PROVIDED FOR CONTROL OF EQUIPMENT, AS NECESSARY, TO MAINTAIN PERSONNEL AND REACTOR SAFETY AND TO AVOID UNAUTHORIZED OPERATION OF EQUIPMENT. PMI 2140 REQUIRES THAT LIFTED LEADS SHALL BE TAGGED TO FOR IDENTIFICATION. CONTRARY TO THE ABOVE, THE INSPECTOR FOUND THAT LEADS OF THE INSTRUMENT AND CONTROL CABLES FOR THE HOT SHUTDOWN PANEL WERE NOT BEING CONTROLLED AS REQUIRED.
(8001 INFRACTION)

10 CFR 50, APPENDIX B, CRITERION XVIII STATES THAT A COMPREHENSIVE SYSTEM OF PLANNED AND PERIODIC AUDITS SHALL BE CARRIED OUT TO VERIFY COMPLIANCE WITH ALL ASPECTS OF THE QUALITY ASSURANCE PROGRAM AND TO DETERMINE THE EFFECTIVENESS OF THE PROGRAM. CONTRARY TO THE ABOVE, THE INSPECTOR FOUND THAT A COMPREHENSIVE SYSTEM OF AUDITS FOR SAFETY RELATED ACTIVITIES AT THE CORPORATE OFFICE HAD NOT BEEN ADDRESSED IN THE AEPSC QUALITY ASSURANCE PROGRAM AND THAT FORMAL AUDITS OF CORPORATE OFFICE ACTIVITIES HAD NOT BEEN IMPLEMENTED.
(8001 INFRACTION)

10 CFR 50, APPENDIX B, CRITERION V STATES THAT ACTIVITIES EFFECTING QUALITY SHALL BE PRESCRIBED BY DOCUMENTED INSTRUCTIONS AND PROCEDURES AND SHALL BE ACCOMPLISHED IN ACCORDANCE WITH THESE INSTRUCTIONS AND PROCEDURES. PMI 7020 REQUIRES THAT IF CORRECTIVE ACTION IS REQUIRED AS PART OF AN AUDIT REPORT, ACTION SHALL BE TAKEN WITHIN 30 DAYS. CONTRARY TO THE ABOVE, CORRECTIVE ACTION FOR AUDIT REPORTS QA 78-12, 78-19 AND 78-21 WAS NOT TAKEN IN THE REQUIRED 30 DAY PERIOD.
(8001 INFRACTION)

TECHNICAL SPECIFICATION 6.8.1 REQUIRES IMPLEMENTATION OF PROCEDURES. 12-OHP-4021.018.004 "DRAINING OR ADJUSTING LEVEL IN THE SPENT FUEL PIT" DETAILS THE PROCEDURE FOR RAISING LEVEL IN THE SFP. CONTRARY TO THE ABOVE, THIS PROCEDURE WAS NOT USED FOR INCREASING LEVEL IN THE SFP BETWEEN MARCH 15, 1980 AND MARCH 17, 1980, WHEN THE POOL WAS OVERFILLED AND CROSS-CONTAMINATION OF THE DEMINERALIZED WATER SYSTEM OCCURRED.
(8003 INFRACTION)

AMENDMENT NUMBER 32 TO NRC OPERATING LICENSE NUMBER DPR-58 AND AMENDMENT NUMBER 13 TO DPR-74 REQUIRE A MINIMUM OF 2000 PPM BORON CONCENTRATION IN THE SPENT FUEL PIT DURING CHANGEOUT FROM OLD RACKS TO NEW HIGH DENSITY FUEL STORAGE RACKS. CONTRARY TO THE ABOVE, ON MARCH 17, 1980, THE SFP CONCENTRATION WAS FOUND TO BE LESS THAN 2000 PPM (1954 PPM).
(8003 INFRACTION)

AS REQUIRED BY 10 CFR 71.5, SHIPMENTS OF LICENSED RADIOACTIVE MATERIAL MUST COMPLY WITH THE REQUIREMENTS OF THE DEPARTMENT OF TRANSPORTATION, 49 CFR 170-189, WHICH REQUIRES PLACARDING OF EACH END AND SIDE OF EXCLUSIVE USE VEHICLES TRANSPORTING LSA MATERIAL. (PARAGRAPH 49 CFR 172.504, 172.506, AND 173.392(C)(7)). CONTRARY TO THE ABOVE, ON FEBRUARY 29, 1980, THREE EXCLUSIVE USE TRUCKS CARRYING LARGE BOXES OF LSA MATERIAL FROM THE D. C. COOK SITE TO THE NUCLEAR ENGINEERING COMPANY, BEATTY, NEVADA, WERE FOUND UPON ARRIVAL TO BE MISSING THE FRONT PLACARD.
(8008 DEFICIENCY)

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-04/ 03L-0	02/02/80	02/27/80	"B" turbine stop valve proximity switch failure.
80-05/ 03L-0	11/13/79	02/28/80	Instrument drift on steam line pressure transmitters.
80-06/ 03L-0	01/29/80	02/28/80	Pressurizer pressure bistable drift.
80-09/ 03L-0	02/08/80	03/29/80	Steam generator level transmitter erratic operation.
80-22/ 03L-0	05/03/80	06/02/80	During normal operation, radiation monitor R-11 alarmed. The sample pump for R-11 and R-12 would not remain running.
80-23/ 03L-0	05/1[illegible]/80	06/10/80	During normal operation, a low flow alarm for the containment APD (R-11/R-12) was received.
80-24/ 03L-0	05/20/80	06/19/80	With the turbine drive aux. feedwater pump in the non-running auto stand-by position, an alarm was received at 1013 hours that the pump had tripped.

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

- Plant shutdown on June 27, 1980, to begin a scheduled outage for modifications to the auxiliary feedwater system.

- LAST IE SITE INSPECTION DATE: February 29, 1980

- INSPECTION REPORT NO: 80-08

SUMMARY

INSPECTION SUMMARY

- Meeting on April 15, (80-06): This special management meeting was held to discuss NRC concerns in two areas: implementation of the American Electric Company Corporate Quality Assurance Program; and the D. C. Cook Plant regulatory performance in the areas of personnel error and adherence to procedure. No items of noncompliance were identified as a result of this meeting.

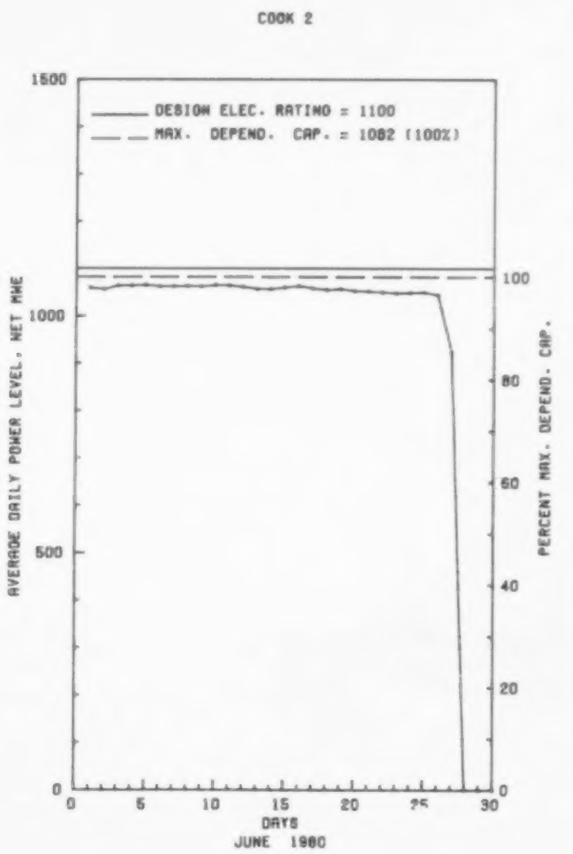
- Inspection on February 29, (80-08): As a part of the inspection conducted during February 25 through March 7, 1980, at Beatty, Nevada, this report details the inspection of trailers No. 232214, No. 331143, and No. 331011, all from D. C. Cook. The shipments consisted of large wooden boxes on an open flatbed trailer. Three apparent items of noncompliance (incomplete placarding) with DOT/NRC regulations were observed.

REVIEWED BY OMPA LOEB: Philip A. Ross **DATE:** 08/04/80

REVIEWED BY NRR DL: **DATE:** 8/5/80

COOK 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-316
2. Utility Contact: W. T. Gillett (616) 465-5901
3. Licensed Thermal Power (MWt): 3391
4. Nameplate Rating (Gross MWe): 1333 x 0.85 = 1133
5. Design Electrical Rating (Net MWe): 1100
6. Maximum Dependable Capacity (Gross MWe): 1118
7. Maximum Dependable Capacity (Net MWe): 1082
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	4,367.0	21,887.3
12. Hours Reactor Was Critical	647.1	3,828.6	14,955.5
13. Reactor Reserve Shutdown Hours	0	.0	.0
14. Hours Generator On-Line	647.1	3,765.5	14,245.6
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	2,187,111	12,194,527	44,418,487
17. Gross Electrical Energy (MWH)	706,465	3,999,020	14,153,430
18. Net Electrical Energy (MWH)	682,148	3,858,710	13,626,122
19. Unit Service Factor	89.9	85.7	73.7
20. Unit Availability Factor	89.9	85.7	73.7
21. Unit Capacity Factor (MDC Net)	87.6	81.7	68.1
22. Unit Capacity Factor (DER Net)	86.1	80.3	67.0
23. Unit Forced Outage Rate	.0	2.1	10.6

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 07/12/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type¹	Duration (Hours)	Reason²	Method of Shutting Down Reactor³	Licensee Event Report No.	System Code⁴	Component Code⁵	Cause & Corrective Action to Prevent Recurrence
78	06/27/80	S	72.9	H	1		CH	PIPEXX	UNIT REMOVED FROM SERVICE FOR SCHEDULED OUTAGE. THE PRIMARY PURPOSE FOR THE OUTAGE IS TO COMPLETE THE AUXILIARY FEEDWATER SYSTEM MODIFICATION WHICH WILL GIVE EACH UNIT ITS INDEPENDENT AUXILIARY FEEDWATER SYSTEM.

SUMMARY

THE UNIT RAN NORMALLY FOR THE REPORTING PERIOD EXCEPT FOR AN OUTAGE ON 06/27/80 FOR FEEDWATER SYSTEM MODIFICATION.

LEGEND

¹Type:
F--Forced
S--Scheduled

²Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

³Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

⁴Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

⁵Exhibit H:
Same source.

]

COOPER STATION

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1. Location (state/county) (nearest population center)	Nebraska/Nemaha 23 miles S of Nebraska City
2. Electric Reliability Council	Mid-Continent Area Reliability Coordination Agreement
3. Type of Reactor	BWR
4. Power Level (MWT)	2381
5. Design Electrical Rating (MWE Net)	778
6. Date of Initial Criticality	February 21, 1974
7. Date of Commercial Operation	July 1, 1974 (50% rating)
8. Condenser Cooling Method	Once Through
9. Condenser Cooling Water Source	Missouri River

Utility & Contractor Information

10. Licensee	Nebraska Public Power District
11. Corporate Address	P. O. Box 499 Columbus, Nebraska 68601
12. Corporate Contact	J. M. Pilant, Director Licensing and Quality Assurance
13. Architect/Engineer	Burns and Roe, Inc.
14. Nuclear Steam Supply System	GE
15. Constructor	Burns and Roe, Inc.

Regulatory Information

16. IE Region responsible	IV
17. IE Branch Chief	G. L. Madsen
18. IE Section Chief	T. F. Westerman
19. IE Resident Inspector	R. G. Spangler
20. Licensing Project Manager	V. L. Rooney
21. Docket Number	50-298
22. License Number/Date of Issue	DPR-46, 1/18/74
23. Public Document Room	Auburn Public Library 118 15th Street Auburn, Nebraska 68305

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

FACILITY ITEMS (PLANS AND PROCEDURES):

MANAGERIAL ITEMS:

PLANT STATUS:

OPERATING

LAST IE SITE INSPECTION DATE: JUNE 3 - 6, AND 23 - 26, 1980

INSPECTION REPORT NO: 50-298/80-09

SUMMARY

INSPECTION SUMMARY

INSPECTION ON JUNE 3-6, 23-26, 1980 (80-09): ROUTINE, UNANNOUNCED INSPECTION OF FOLLOWUP OF PREVIOUSLY IDENTIFIED ITEMS, REVIEW OF PLANT OPERATIONS REFUELING, SHUTDOWN MARGIN DETERMINATION, CALIBRATION, IEB FOLLOWUP AND INDEPENDENT INSPECTION EFFORT. WITHIN THESE 6 AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE WERE FOUND IN 5 AREAS. ONE ITEM OF NONCOMPLIANCE WAS IDENTIFIED DURING THE INDEPENDENT INSPECTION EFFORT (DEFICIENCY - FAILURE TO PROPERLY CONTROL COMBUSTIBLE MATERIALS).

REPORTS RECEIVED FROM LICENSEE

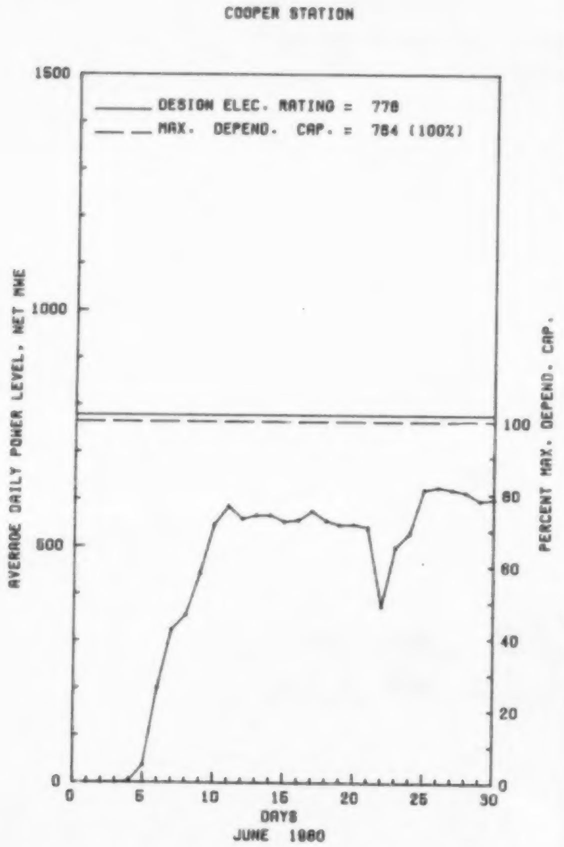
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
	NONE		

REVIEWED BY OMPA LOEB: **DATE:** 8-4-80

REVIEWED BY NRR DL: **DATE:** 8/4/80

COOPER STATION

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-298
2. Utility Contact: P. J. Borer (402) 825-3811
3. Licensed Thermal Power (MWt): 2381
4. Nameplate Rating (Gross MWe): 983 x 0.85 = 835
5. Design Electrical Rating (Net MWe): 778
6. Maximum Dependable Capacity (Gross MWe): 787
7. Maximum Dependable Capacity (Net MWe): 764
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): 635
10. Reasons for Restrictions, If Any: REPLACED TURBINE BLADES & DISCS

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	4,367.0	52,698.0
12. Hours Reactor Was Critical	658.3	2,105.3	43,342.6
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	616.7	2,054.4	42,530.0
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,211,064	4,288,584	80,471,510
17. Gross Electrical Energy (MWH)	329,828	1,345,023	25,886,871
18. Net Electrical Energy (MWH)	315,966	1,297,094	24,955,371
19. Unit Service Factor	85.7	47.0	80.8
20. Unit Availability Factor	85.7	47.0	83.8
21. Unit Capacity Factor (MDC Net)	57.4	38.9	62.1
22. Unit Capacity Factor (DER Net)	56.4	38.2	61.0
23. Unit Forced Outage Rate	2.7	2.0	4.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-5	03/01/80	S	85.9	C	4		RC	FUELXX	1980 REFUELING AND MAINTENANCE OUTAGE.
80-6	06/06/80	F	17.4	A	3				REACTOR SCRAM FROM LOW REACTOR WATER LEVEL FOLLOWING REACTOR FEEDWATER PUMP OVERSPEED TRIP. MALFUNCTIONING REACTOR FEED PUMP CONTROLLER PROBLEM WAS CORRECTED AND REACTOR RETURNED TO OPERATION.
80-7	06/22/80	S	.0	H	5				POWER WAS REDUCED TO ALLOW ADJUSTMENT OF THE CONTROL ROD PATTERN.

SUMMARY

FOLLOWING A REFUELING OUTAGE BEGUN ON 03/01/80, AND A REACTOR TEST PUMP OUTAGE ON 06/06/80, THE UNIT WAS RETURNED TO SERVICE. IT OPERATED NORMALLY FOR THE REPORTING PERIOD WITH A POWER REDUCTION ON 06/22/80 FOR ADJUSTMENTS TO CONTROL ROD PATTERNS.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

CRYSTAL RIVER 3

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Florida/Citrus 7 miles NW of Crystal River
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2452
5.	Design Electrical Rating (MWE Net)	825
6.	Date of Initial Criticality	January 14, 1977
7.	Date of Commercial Operation	March 13, 1977
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Gulf of Mexico

Utility & Contractor Information

10.	Licensee	Florida Power Corporation
11.	Corporate Address	3201 34th Street, South St. Petersburg, Florida 33733
12.	Corporate Contact	J. Hancock
13.	Architect/Engineer	Gilbert Associates, Inc.
14.	Nuclear Steam Supply System	B&W
15.	Constructor	J. A. Jones Construction Co.

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. Lewis
18.	IE Section Chief	R. Martin
19.	IE Resident Inspector	T. F. Stetka
20.	Licensing Project Manager	P. B. Erickson
21.	Docket Number	50-302
22.	License Number/Date of Issue	DPR-72, 12/3/76
23.	Public Document Room	Crystal River Public Library 668 N.W. First Crystal River, Florida 32629

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

ANTICIPATED CRITICALITY FOLLOWING OUTAGE 7114180.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

REFUELING SHUTDOWN.

LAST IE SITE INSPECTION DATE: MAY 5-30, 1980

INSPECTION REPORT NO: 50-302/80-23

SUMMARY

INSPECTION SUMMARY

INSPECTION MAY 5-30 (80-23): THIS ROUTINE INSPECTION BY THE RESIDENT INSPECTOR OF PLANT OPERATIONS, SECURITY, AND RADIOLOGICAL CONTROLS, LICENSEE EVENT REPORTS (LER'S), LICENSEE ACTION ON IE INFORMATION NOTICES, NONROUTINE EVENTS, SURVEILLANCES TESTS, AND TOUR OF PLANT AREAS. THE INSPECTORS ALSO OBSERVED THE LICENSEE'S EMERGENCY PLAN DRILL CONDUCTED ON MAY 29, OBSERVED PLANT OPERATIONS ON THE BACK SHIFT, AND ATTENDED A MEETING WITH NRR PERSONNEL ON MAY 19. THE INSPECTION INVOLVED 129.5 HOURS ON SITE BY TWO RESIDENT INSPECTORS. TWO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED (INFRACTION - FAILURE OF SECURITY GUARDS TO EITHER SEARCH PERSONNEL OR REQUIRE PERSONNEL TO RE-ENTER THE WEAPONS DETECTOR AFTER THE DEVICE HAD ALARMED; INFRACTION - FAILURE OF PERSONNEL TO SELF-FRISK PRIOR TO EXITING THE RCA).

REPORTS RECEIVED FROM LICENSEE

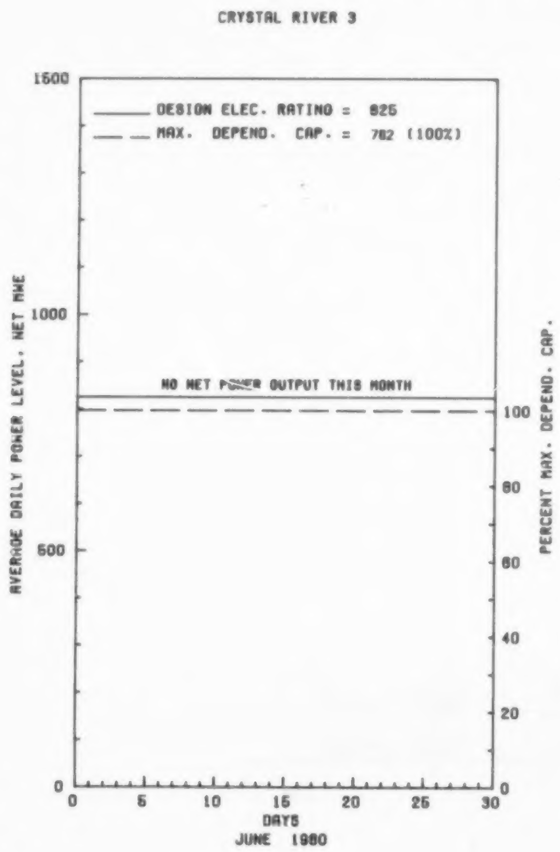
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-019/ 03L-0	05/17/80	05/29/80	CRACKED FUEL ASSEMBLY HOLDDOWN SPRING.
80-020/ 03L-0	05/01/80	05/29/80	FAILURE TO PERFORM WEEKLY BATTERY CHECK SURVEILLANCE WITHIN ALLOWED INTERVAL.
80-021/ 01T-0	05/06/80	05/29/80	FEEDWATER ISOLATION VALVE INOPERABLE DUE TO MISSING HINGE PIN.
80-022/ 04L-0	05/28/80	06/06/80	ENVIRONMENTAL WATER SAMPLES MISSED.
80-023/ 03L-0	05/17/80	06/12/80	CONTAMINATED WASTE TAKEN TO SITE DUMP.

REVIEWED BY OMPA LOEB: PE Bote **DATE:** 8/4/80

REVIEWED BY NRR DL: P B Erickson **DATE:** 8/5/80

CRYSTAL RIVER 3

AVERAGE POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-302
2. Utility Contact: M.W. CULVER (904) 795-6486
3. Licensed Thermal Power (MWt): 2452
4. Nameplate Rating (Gross MWe): 989 x 0.9 = 890
5. Design Electrical Rating (Net MWe): 825
6. Maximum Dependable Capacity (Gross MWe): 821
7. Maximum Dependable Capacity (Net MWe): 782
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	4,367.0	28,943.0
12. Hours Reactor Was Critical	.0	1,305.5	16,455.7
13. Reactor Reserve Shutdown Hours	.0	.0	1,116.9
14. Hours Generator On-Line	.0	1,297.4	15,997.7
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	3,085,927	35,402,838
17. Gross Electrical Energy (MWH)	0	1,058,296	12,030,164
18. Net Electrical Energy (MWH)	0	1,009,335	11,400,975
19. Unit Service Factor	.0	29.7	55.3
20. Unit Availability Factor	.0	29.7	55.3
21. Unit Capacity Factor (MDC Net)	.0	29.6	50.4
22. Unit Capacity Factor (DER Net)	.0	28.0	47.7
23. Unit Forced Outage Rate	.0	18.0	32.0

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): CONTINUING REFUELING OUTAGE.
25. If Shut Down At End of Report Period, Estimated Date of Startup: 08/06/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-9	03/07/80	S	720.0	C	4		RC	FUELXX	CONTINUATION OF THE REFUELING OUTAGE.

SUMMARY

THE UNIT WAS SHUTDOWN FOR REFUELING AND MAINTENANCE OUTAGE.

LEGEND

[1] Type:
F-Forced
S-Scheduled

[2] Reason:
A-Equipment Failure
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error
H-Other

[3] Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Continuations
5-Load Reduction
9-Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

NRC POWER RESTRICTIONS: NONE

DAVIS-BESSE 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Ohio/Ottawa 21 miles E of Toledo
2.	Electric Reliability Council	East Central Area Reliability Coordination Agreement
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2772
5.	Design Electrical Rating (MWE Net)	906
6.	Date of Initial Criticality	August 12, 1977
7.	Date of Commercial Operation	11/21/77 (25%); 12/19/77 (40%) 01/23/78 (75%); 07/31/78 (100%)
8.	Condenser Cooling Method	Cooling Tower
9.	Condenser Cooling Water Source	Lake Erie

Utility & Contractor Information

10.	Licensee	Toledo Edison Co.
11.	Corporate Address	Edison Plaza, 300 Madison Avenue Toledo, Ohio 43652
12.	Corporate Contact	R. P. Crouse, V. P., Nuclear (419)259-5221
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	B&W
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	R. F. Warnick
19.	IE Resident Inspector	L. Reyes (419) 898-2775
20.	Licensing Project Manager	D. Garner
21.	Docket Number	50-346
22.	License Number/Date of Issue	NPF-3, 4/22/77
23.	Public Document Room	Ida Rupp Public Library 310 Madison Street Port Clinton, OH 43452

INSPECTION STATUS

ENFORCEMENT STATUS

THE LICENSEE FAILED TO ADEQUATELY SEARCH SOME INDIVIDUALS PRIOR TO THEIR ENTRY INTO THE PROTECTED AREA.
(8013 INFRACTION)

THE LICENSEE FAILED TO PROPERLY ESCORT SEVERAL INDIVIDUALS.
(8013 INFRACTION)

A VITAL AREA DOOR WAS FOUND UNSECURED AND UNATTENDED AND SEVERAL WATCHMEN WERE NOT PROVIDED WITH ADEQUATE COMMUNICATION.
(8013 INFRACTION)

THE LICENSEE DID NOT CONDUCT PROPER PERSONNEL SCREENING IN ACCORDANCE WITH SECURITY PLAN COMMITMENTS.
(8013 INFRACTION)

THE LICENSEE FAILED TO PERFORM A REQUIRED TEST ON AN ALARM SYSTEM.
(8013 DEFICIENCY)

TECHNICAL SPECIFICATION 3.3.3.8 REQUIRES THE FIRE DETECTION INSTRUMENTATION TO BE OPERABLE AT ALL TIMES, OR TO ESTABLISH WITHIN ONE HOUR A FIRE WATCH PATROL. CONTRARY TO THE ABOVE, ON APRIL 7, 1980, POWER WAS LOST TO THE FIRE DETECTION INSTRUMENTATION AT 1400 HOURS AND A FIRE WATCH PATROL WAS NOT ESTABLISHED UNTIL APPROXIMATELY 1700 HOURS.
(8014 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

+ The refueling/maintenance outage continued during the month. In reference to the broken fuel assembly holddown spring problem discovered May 1980, the licensee has initiated a spring replacement program. Forty-four springs have been replaced to date. The current outage is expected to last until the later part of July 1980.

+LAST IE SITE INSPECTION DATE: May 12-16, 19-23, 28-30; June 2-6 and 9-10, 1980

+ INSPECTION REPORT NO: 80-15

SUMMARY

INSPECTION SUMMARY

+ Inspection on December 18-19, (79-34): Nonroutine, announced inspection of licensee actions taken in response to IE Bulletin 79-19, "Packaging of Low-Level Radioactive Waste for Transport and Burial." The inspection involved 14 inspector hours onsite by one NRC inspector. No items of noncompliance or deviations were identified.

+ Inspection on April 10-11, 21-22; May 6-7 and June 4, (80-11): Quality Assurance/Quality Control program, implementing procedures, observation of work activities and review and evaluate nondestructive examinations (NDE) data and reports of inservice inspection (ISI); IE Bulletin 79-13 and IE Bulletin 79-17 activities. The inspection involved 22 inspector hours onsite by one NRC inspector. No items of noncompliance or deviations were identified.

+ Inspection on April 8-11, 15-25, 29-30; May 1-2, and 5-9, (80-14): Routine unannounced inspection of long term shutdown, IE Bulletin followup, special inspection of the April 19, 1980 loss of decay heat removal system and instrument power; Part 21, Report and License Event Report. The inspection involved 319.5 inspector hours onsite by three NRC inspectors. Of the five areas inspected, no items of noncompliance or deviations were found in four areas; one item of apparent noncompliance was identified in the other area (failure to establish a fire watch patrol).

+ Inspection on May 12-16, 19-23, 26-30; June 2-6 and 9-10, (80-15): Routine unannounced inspection of followup on previous inspection findings, long term shutdown, IE Circular followup and licensee event report. The inspection involved 129 inspector hours onsite during regular shift and 29 inspector hours during offshifts by the resident inspector. Of the four areas inspected, no items of noncompliance or deviations were found in three areas; one item of apparent noncompliance was identified in the other area (failure to notify the NRC Operations Center as required by 10 CFR 50.72).

REPORTS RECEIVED FROM LICENSEE

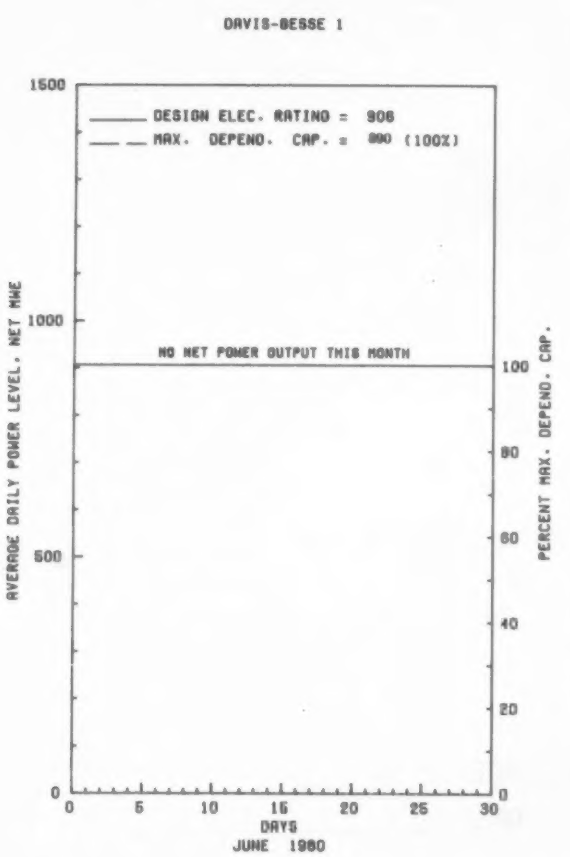
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-37/ 03L-0	04/18/80	05/16/80	Steam generator level transmitter out of calibration.
80-39/ 03L-0	04/23/80	05/21/80	Weekly test late by 6 hours and 10 minutes.
80-40/ 03L-0	05/14/80	05/23/80	Holddown spring on fuel assembly found to be damaged.
80-41/ 03L-0	05/11/80	06/05/80	Source range channel out of tolerance and declared inoperable.
80-42/ 03L-0	05/10/80	06/06/80	Combined leakage for all penetrations exceeded allowable leakage.

REVIEWED BY OMPA LOEB: ______ **DATE:** 8/4/80

REVIEWED BY NRR DL: ______ **DATE:** 8/4/80

DAVIS-BESSE 1

AVERAGE POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-346
2. Utility Contact: BILAL SARSOUR (419) 259-5000 X236
3. Licensed Thermal Power (MWt): 2772
4. Nameplate Rating (Gross MWe): 1069 x 0.9 = 962
5. Design Electrical Rating (Net MWe): 906
6. Maximum Dependable Capacity (Gross MWe): 934
7. Maximum Dependable Capacity (Net MWe): 890
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	4,367.0	24,892.0
12. Hours Reactor Was Critical	.0	2,078.2	13,042.4
13. Reactor Reserve Shutdown Hours	.0	.0	2,875.8
14. Hours Generator On-Line	.0	2,008.7	11,883.4
15. Unit Reserve Shutdown Hours	.0	.0	1,728.2
16. Gross Thermal Energy (MWH)	0	4,687,305	24,886,811
17. Gross Electrical Energy (MWH)	0	1,583,554	8,307,070
18. Net Electrical Energy (MWH)	0	1,483,787	7,654,365
19. Unit Service Factor	.0	46.0	48.8
20. Unit Availability Factor	0	46.0	56.4
21. Unit Capacity Factor (MDC Net)	.0	38.2	37.1
22. Unit Capacity Factor (DER Net)	.0	37.5	36.4
23. Unit Forced Outage Rate	.0	14.3	25.6

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 07/28/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
4.	04/07/80	S	720.0	C	4				UNIT OUTAGE WHICH BEGAN ON APRIL 7, 1980, WAS STILL IN PROGRESS THROUGH THE END OF JUNE, 1980.

SUMMARY

THE UNIT REMAINED SHUTDOWN IN JUNE FOR REFUELING AND MAJOR MODIFICATIONS ORDERED BY NUREG 0578 (TMI LESSONS LEARNED).

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

best copy available

DRESDEN 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Illinois/Grundy 9 miles E of Morris
2.	Electric Reliability Council	Mid-America Interpool Network
3.	Type of Reactor	BWR
4.	Power Level (MWT)	700
5.	Design Electrical Rating (MWE Net)	200
6.	Date of Initial Criticality	October 15, 1959
7.	Date of Commercial Operation	July 4, 1960
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Kankakee River

Utility & Contractor Information

10.	Licensee	Commonwealth Edison Co.
11.	Corporate Address	P. O. Box 767 Chicago, Illinois 60690
12.	Corporate Contact	C. Reed
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	GE
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	R. Spessard
19.	IE Resident Inspector	J. L. Barker (815)-942-9267
20.	Licensing Project Manager	Paul W. O'Connor
21.	Docket Number	50-010
22.	License Number/Date of Issue	DPR-2 issued 9/28/59
23.	Public Document Room	Morris Public Library 604 Liberty Street Morris, Illinois 60451

INSPECTION STATUS

ENFORCEMENT STATUS

10 CFR 20.201(3) REQUIRES THAT EACH LICENSEE SHALL MAKE OR CAUSE TO BE MADE SUCH SURVEYS AS MAY BE NECESSARY TO COMPLY WITH THE REGULATIONS IN PART 20. CONTRARY TO THIS REQUIREMENT, NO TIMELY SURVEY (EVALUATION) WAS MADE UPSTREAM OF THE COMPOSITE SAMPLER IN THE DISCHARGE CANAL ON APRIL 5, 1980, AFTER AN UNPLANNED LIQUID RELEASE FROM UNIT 1, TO ENSURE COMPLIANCE WITH 10 CFR 20.106(A).
(8008 INFRACTION)

TECHNICAL SPECIFICATION 3.8.C.2 STATES, IN PART, THAT CONCENTRATIONS OF GROSS BETA ACTIVITY IN THE CONDENSER COOLING WATER DISCHARGE CANAL SHALL NOT EXCEED 1E-7 MICROCURIES PER MILLILITER UNLESS THE DISCHARGE IS CONTROLLED IN ACCORDANCE WITH 10 CFR 20 LIMITS. CONTRARY TO THIS REQUIREMENT, AS THE RESULT OF AN UNCONTROLLED RELEASE, CONCENTRATIONS OF 1.8E-6 AND 3.7E-6 MICROCURIES PER MILLILITER OF GROSS BETA ACTIVITY EXISTED IN THE DISCHARGE CANAL ON APRIL 5 AND 6, 1980.
(8008 INFRACTION)

10 CFR 50, APPENDIX B, CRITERION II REQUIRES ACTIVITIES AFFECTING QUALITY BE ACCOMPLISHED UNDER SUITABLY CONTROLLED CONDITIONS, INCLUDING ADEQUATE CLEANNESS. THE LICENSEE'S QUALITY ASSURANCE PROGRAM, SECTION 2.2 REQUIRES THAT THE LICENSEE ADHERE TO ALL MANDATORY REQUIREMENTS OF ANSI N18.7. ANSI N18.7-1976, SECTION 5.2.10 REQUIRES QUALITY HOUSEKEEPING PRACTICES ENCOMPASSING ALL ACTIVITIES RELATED TO CONTROL OF FIRE PREVENTION AND PROTECTION, INCLUDING DISPOSAL OF COMBUSTIBLE MATERIAL AND DEBRIS. CONTRARY TO THE ABOVE, ON APRIL 17, 1980, DURING A ROUTINE TOUR OF THE UNIT 1 SPHERE, THE NRC INSPECTOR OBSERVED NUMEROUS OILY RAGS/PAPERS, A TIPPED OVER LUBE OIL CAN, AND SCATTERED DEBRIS ABOVE THE ELEVATOR SHAFT WHICH WERE NOT BEING CONTROLLED AND WHICH REPRESENTED A FIRE HAZARD.
(8009 INFRACTION)

UNIT 1 TECHNICAL SPECIFICATIONS, SECTION 6.2.B REQUIRES THAT RADIATION CONTROL PROCEDURES BE MAINTAINED, MADE AVAILABLE TO ALL STATION PERSONNEL, AND ADHERED TO. RADIATION CONTROL STANDARDS PROCEDURE 37-1-E-3, "WORK IN CONTROLLED AREAS (RADIATION AREAS AND HIGH RADIATION AREAS)," REQUIRES THAT PERSONNEL NOT EAT, DRINK, SMOKE, OR CHEW IN THOSE CONTROLLED AREAS. CONTRARY TO THE ABOVE, ON APRIL 17, 1980, WHILE MAKING A ROUTINE TOUR OF THE UNIT 1 TURBINE BUILDING (A POSTED RADIATION AREA), THE NRC INSPECTOR OBSERVED EVIDENCE OF EATING, DRINKING, AND SMOKING (I.E., THE PRESENCE OF NUMEROUS CIGARETTE BUTTS, EMPTY SOFT DRINK CANS, EMPTY CANDY WRAPPERS, AND A HALF EATEN HAMBURGER) IN THIS RADIATION AREA. THIS IS A REPETITIVE ITEM OF NONCOMPLIANCE SINCE THE SAME PROBLEM WAS IDENTIFIED TWICE PREVIOUSLY IN NRC INSPECTION REPORTS NO. 50-010/79-19, DATED OCTOBER 18, 1979, AND NO. 50-010/79-25, DATED JANUARY 28, 1980.
(8009 INFRACTION)

UNIT 1 TECHNICAL SPECIFICATIONS, SECTION 6.2.B REQUIRES THAT RADIATION CONTROL PROCEDURES BE MAINTAINED, MADE AVAILABLE TO ALL STATION PERSONNEL, AND ADHERED TO. RADIATION CONTROL STANDARDS PROCEDURE 37-1-A-1 REQUIRES THAT CONTAMINATED CLOTHING SHOULD BE REMOVED FROM CONTROLLED CONTAMINATED AREAS WHEN NOT IN USE AND, FURTHER, REQUIRES THAT CLOTHING HAMPERS MARKED "DEPOSIT CONTAMINATED RUBBER GOODS HERE" AND "DEPOSIT CONTAMINATED CANVAS GOODS HERE" BE PLACED AT THE EXITS FROM ALL AREAS WHERE PROTECTIVE CLOTHING IS REQUIRED. CONTRARY TO THE ABOVE, ON APRIL 17, 1980, DURING A ROUTINE TOUR OF THE UNIT 1 TURBINE BUILDING, THE NRC INSPECTOR OBSERVED CONTAMINATED CLOTHING LYING INSIDE A CONTROLLED AREA (UNIT 1 CONDENSATE DEMINERALIZER CONTROL AREA) AND THAT NO CLOTHING HAMPERS WERE LOCATED AT THE EXIT OF THIS AREA. THIS CONDITION WAS DETERMINED TO HAVE EXISTED FOR A PERIOD OF TWO WEEKS.
(8009 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

Unit 1 shutdown on October 31, 1978 for an extended outage to upgrade ECCS and Reactor Protection System chemical cleaning and refueling. Outage expected to last through Mid-1986.

+LAST IE SITE INSPECTION DATE: May 5-29, 1950

+ INSPECTION REPORT NO: 80-10

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
NONE				

SUMMARY

INSPECTION SUMMARY

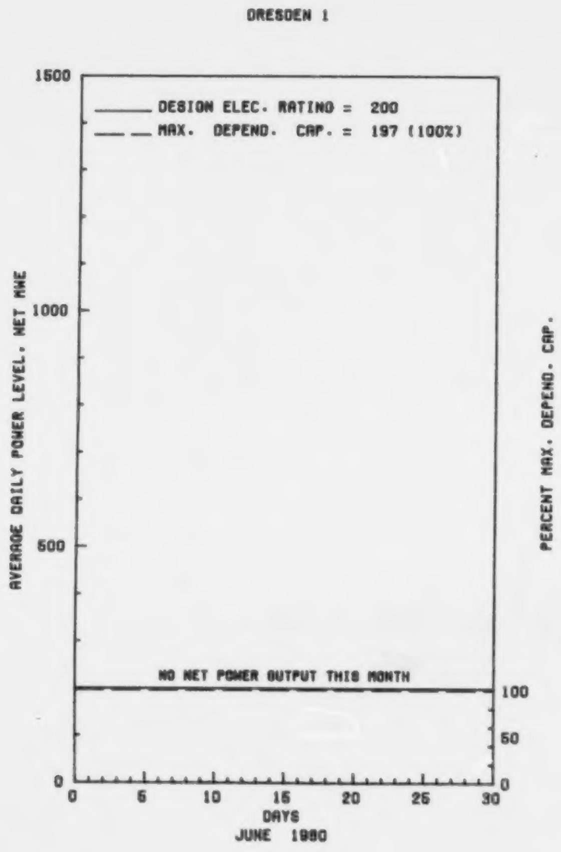
Inspection on May 5-29, (80-10): Routine, resident inspection of licensee action on previous inspection findings, operational safety verification, licensee event reports, plant trips, inspection during long term shutdown, and outstanding inspection items. The inspection involved 97 inspector hours onsite by one NRC inspector. Of the six areas inspected, there were no items of noncompliance identified in five areas. There was one item of noncompliance (failure to properly report prompt reportable events in accordance with Technical Specifications) identified in one area.

REVIEWED BY OMPA LOEB: **DATE:** 8-4-80

REVIEWED BY NRR DL: Paul W O'Connor **DATE:** 8/5/80

DRESDEN 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-010
2. Utility Contact: S. A. Cieszkiewicz (815) 942-2920
3. Licensed Thermal Power (MWt): 700
4. Nameplate Rating (Gross MWe): 245.5 x 0.85 = 209
5. Design Electrical Rating (Net MWe): 200
6. Maximum Dependable Capacity (Gross MWe): 205
7. Maximum Dependable Capacity (Net MWe): 197
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	4,367.0	177,167.0
12. Hours Reactor Was Critical	.0	.0	113,153.9
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	.0	.0	111,196.2
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	0	56,014,747
17. Gross Electrical Energy (MWH)	0	0	16,799,256
18. Net Electrical Energy (MWH)	-1,336	-6,045	15,803,782
19. Unit Service Factor	.0	.0	62.8
20. Unit Availability Factor	.0	.0	62.8
21. Unit Capacity Factor (MDC Net)	.0	.0	45.3
22. Unit Capacity Factor (DER Net)	.0	.0	44.6
23. Unit Forced Outage Rate	.0	.0	11.2

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
8	10/31/78	S	720.0	H	4				REFUELING AND CHEMICAL CLEANING OUTAGE.

SUMMARY

THE UNIT REMAINED SHUTDOWN THROUGHOUT THE REPORTING PERIOD FOR REFUELING AND CHEMICAL CLEANING.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

NRC POWER RESTRICTIONS: NONE

DRESDEN 2

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Illinois/Grundy 9 miles E of Morris
2.	Electric Reliability Council	Mid-America Interpool Network
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2527
5.	Design Electrical Rating (MWE Net)	794
6.	Date of Initial Criticality	January 7, 1970
7.	Date of Commercial Operation	JULY, 1970
8.	Condenser Cooling Method	Cooling Lake
9.	Condenser Cooling Water Source	Kankakee and Des Plaines Rivers

Utility & Contractor Information

10.	Licensee	Commonwealth Edison Co.
11.	Corporate Address	P. O. Box 767 Chicago, Illinois 60690
12.	Corporate Contact	C. Reed
13.	Architect/Engineer	Sargent & Lundy
14.	Nuclear Steam Supply System	GE
15.	Constructor	United Engineers & Constructors, Inc.

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	R. Spessard
19.	IE Resident Inspector	J. L. Barker (815)-942-9267
20.	Licensing Project Manager	Paul W. O'Connor
21.	Docket Number	50-237
22.	License Number/Date of Issue	DPR-19 issued 12/22/69
23.	Public Document Room	Morris Public Library 604 Liberty Street Morris, Illinois 60451

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

Unit operating routinely.

+LAST IE SITE INSPECTION DATE: May 5-29, 1980

+ INSPECTION REPORT NO: 80-09

SUMMARY

INSPECTION SUMMARY

+ Inspection on May 5-29, (80-09) Routine, resident inspection of licensee action on previous inspection findings, operational safety verification, licensee event reports, plant trips, inspection during long term shutdown, and outstanding inspection items. The inspection involved 97 inspector hours onsite by one NRC inspector. Of the six areas inspected, there were no items of noncompliance identified in five areas. There was one item of noncompliance (failure to properly report prompt reportable events in accordance with Technical Specifications) identified in one area.

REPORTS RECEIVED FROM LICENSEE

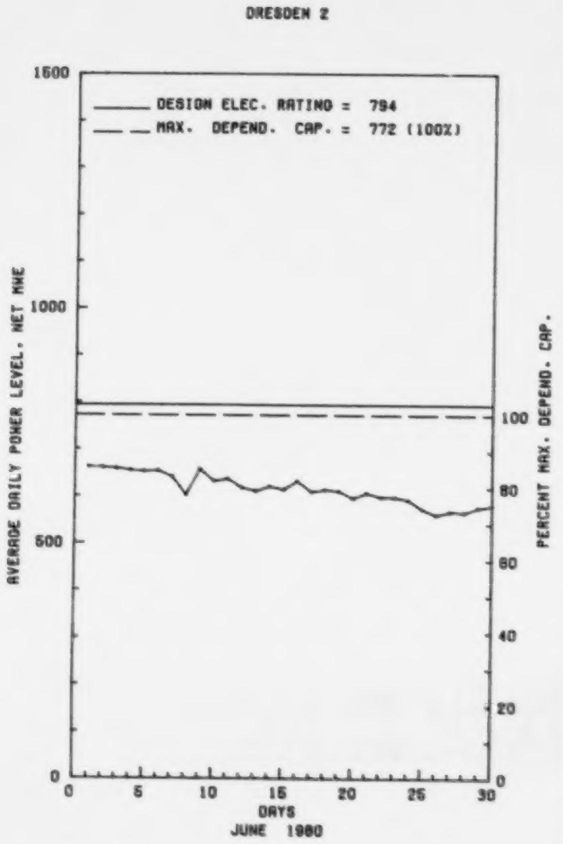
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-14/ 03L-0	04/29/80	05/27/80	During normal operation, 2C main steam line rad monitor caused RPS channel "A" half-scram.
80-15/ 03L-0	05/02/80	05/28/80	During steady state operation, torus to drywell vacuum breaker 2-1601-33A was inoperable during routine oper. surv.
80-16/ 01T-0	05/15/80	05/21/80	While shutdown and conducting 300 psi primary system hydrostatic leak inspect. water dripped from LPCI check valve upstream drain line.
80-17/ 03L-0	05/12/80	06/05/80	Following a unit scram, double valve indication was observed on the HPCI inboard isolation valve.
80-18/ 03L-0	05/12/80	06/03/80	During reactor startup, HPCI inboard steam isolation valve failed to close (TS 3.7.D.1).

REVIEWED BY OMPA LOEB: John J. Rizzo DATE: 8-4-80

REVIEWED BY NRR DL: ______ DATE: ______

DRESDEN 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-237
2. Utility Contact: S. A. CIESZKIEWICZ (815) 942-2920 X489
3. Licensed Thermal Power (MWt): 2527
4. Nameplate Rating (Gross MWe): 920 x 0.9 = 828
5. Design Electrical Rating (Net MWe): 794
6. Maximum Dependable Capacity (Gross MWe): 812
7. Maximum Dependable Capacity (Net MWe): 772
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	4,367.0	85,823.0
12. Hours Reactor Was Critical	720.0	4,207.0	68,943.7
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	720.0	4,176.6	65,439.3
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,513,022	9,426,630	131,268,462
17. Gross Electrical Energy (MWH)	467,073	3,049,211	42,055,832
18. Net Electrical Energy (MWH)	448,735	2,893,234	39,781,346
19. Unit Service Factor	100.0	95.6	73.7
20. Unit Availability Factor	100.0	95.6	73.7
21. Unit Capacity Factor (MDC Net)	79.3	85.8	58.0
22. Unit Capacity Factor (DER Net)	7[illegible].1	83.4	56.4
23. Unit Forced Outage Rate	.0	1.1	12.7

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	NONE								

SUMMARY

THE UNIT OPERATED NORMALLY THROUGHOUT REPORTING PERIOD.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

NRC POWER RESTRICTIONS: NONE

DRESDEN 3

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Illinois/Grundy 9 miles E of Morris
2.	Electric Reliability Council	Mid-America Interpool Network
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2527
5.	Design Electrical Rating (MWE Net)	794
6.	Date of Initial Criticality	January 31, 1971
7.	Date of Commercial Operation	November 16, 1971
8.	Condenser Cooling Method	Cooling Lake
9.	Condenser Cooling Water Source	Kankakee and Des Plaines Rivers

Utility & Contractor Information

10.	Licensee	Commonwealth Edison Co.
11.	Corporate Address	P. O. Box 767 Chicago, Illinois 60690
12.	Corporate Contact	D. L. Peoples
13.	Architect/Engineer	Sargent & Lundy
14.	Nuclear Steam Supply System	GE
15.	Constructor	United Engineers & Constructors, Inc.

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	R. Spessard
19.	IE Resident Inspector	J. L. Barker 815-942-9267
20.	Licensing Project Manager	R. Bevan
21.	Docket Number	50-249
22.	License Number/Date of Issue	DPR-25 issued 1/12/71
23.	Public Document Room	Morris Public Library 604 Liberty Street Morris, Illinois 60451

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

Operating routinely.

+LAST IE SITE INSPECTION DATE: May 5-29, 1980

+ INSPECTION REPORT NO: 80-13

SUMMARY

INSPECTION SUMMARY

+ Inspection on May 5-29, (80-13): Routine, resident inspection of licensee action on previous inspection findings, operational safety verification, licensee event reports, plant trips, inspection during long term shutdown, and outstanding inspection items. The inspection involved 97 inspector hours onsite by one NRC inspector. Of the six areas inspected, there were no items of noncompliance identified in five areas. There was one item of noncompliance (failure to properly report prompt reportable events in accordance with Technical Specifications) identified in one area.

REPORTS RECEIVED FROM LICENSEE

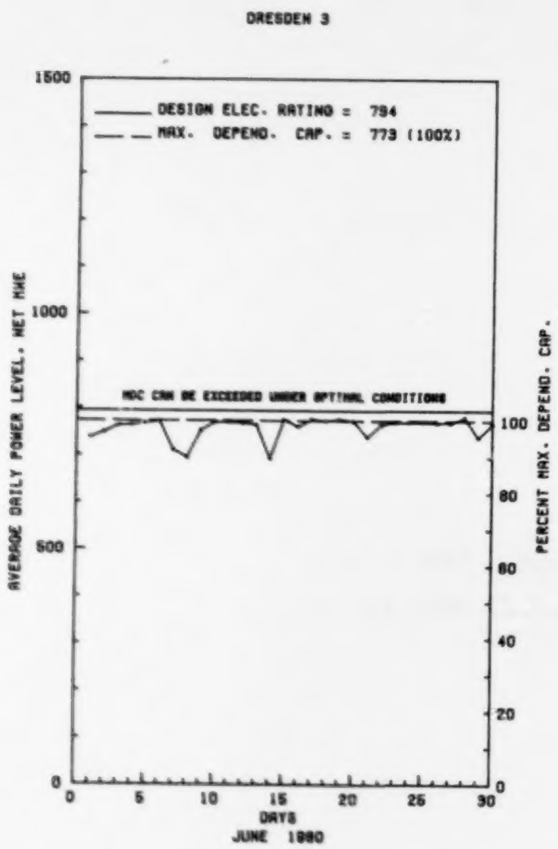
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-22/ 03L-0	04/29/80	05/28/80	During normal shutdown conditions, IRM 12 downscale rod block was found to trip at 4/125 TS requires 5/125.
80-23/ 03L-0	05/19/80	06/12/80	Level switch LIS 3-263-58A tripped at less than the TS limit of 144 inches during level instrument surveillance.
80-24/ 03L-0	05/19/80	06/16/80	Level switch LIS 3-263-58B tripped at less than TS limit of 84 inches with reactor level instrument surveillance in progress.
80-25/ 03L-0	05/21/80	06/11/80	During main steam line high flow isolation surveillance flow switch DPIS 3-261-2A was out of TS limits.

REVIEWED BY OMPA LOEB: John J. Rizzo DATE: 8-4-80

REVIEWED BY NRR DL: RBBevan DATE: 8/5/80

DRESDEN 3

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-249
2. Utility Contact: S. A. Cieszkiewicz (815) 942-2920 x263
3. Licensed Thermal Power (MWt): 2527
4. Nameplate Rating (Gross MWe): 920 x 0.9 = 828
5. Design Electrical Rating (Net MWe): 794
6. Maximum Dependable Capacity (Gross MWe): 812
7. Maximum Dependable Capacity (Net MWe): 773
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	4,367.0	78,408.0
12. Hours Reactor Was Critical	720.0	2,196.2	57,961.6
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	720.0	2,130.6	55,455.2
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,777,019	4,411,011	106,692,679
17. Gross Electrical Energy (MWH)	570,651	1,410,855	34,840,529
18. Net Electrical Energy (MWH)	543,104	1,327,721	33,021,012
19. Unit Service Factor	100.0	48.8	70.7
20. Unit Availability Factor	100.0	48.8	70.7
21. Unit Capacity Factor (MDC Net)	97.6	39.3	54.5
22. Unit Capacity Factor (DER Net)	95.0	38.3	53.0
23. Unit Forced Outage Rate	.0	3.5	16.1

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	NONE								

SUMMARY

THE UNIT OPERATED NORMALLY THROUGHOUT REPORTING PERIOD.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

NRC POWER RESTRICTIONS: NONE

DUANE ARNOLD

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Iowa/Linn 8 miles NW of Cedar Rapids
2.	Electric Reliability Council	Mid-Continent Area Reliability Coordination Agreement
3.	Type of Reactor	BWR
4.	Power Level (MWT)	1658
5.	Design Electrical Rating (MWE Net)	538
6.	Date of Initial Criticality	March 23, 1974
7.	Date of Commercial Operation	February 1, 1975
8.	Condenser Cooling Method	Cooling Towers
9.	Condenser Cooling Water Source	Cedar Rapids River Circulating Water System

Utility & Contractor Information

10.	Licensee	Iowa Electric Light and Power
11.	Corporate Address	IE Towers, P. O. Box 351 Cedar Rapids, Iowa 52406
12.	Corporate Contact	D. Arnold, President
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	GE
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	W. S. LITTLE
19.	IE Resident Inspector	W. F. CHRISTIANSON
20.	Licensing Project Manager	K. Eccleston
21.	Docket Number	50-331
22.	License Number/Date of Issue	DPR-49 issued 2/22/74
23.	Public Document Room	Reference Service Cedar Rapids Public Library 428 Third Avenue, S.E. Cedar Rapids, Iowa 52401

INSPECTION STATUS

ENFORCEMENT STATUS

10 CFR 50, APPENDIX B, CRITERIA XIII, STATES, IN PART, "MEASURES SHALL BE ESTABLISHED TO CONTROL THE...STORAGE...AND PRESERVATION OF MATERIAL AND EQUIPMENT IN ACCORDANCE WITH WORK AND INSPECTION INSTRUCTIONS." QAD 1313.1 STATES, IN PART, "SPECIAL COVERINGS, EQUIPMENT AND PROTECTIVE ENVIRONMENTS...SHALL BE SPECIFIED, PROVIDED, AND THEIR USE VERIFIED WHERE NECESSARY." CONTRARY TO THE ABOVE, ON DECEMBER 11, 1979, THE INSPECTOR OBSERVED THAT THE RECIRCULATION SYSTEM SWITCHGEAR AND ASSOCIATED BREAKERS MARKED FOR LEVEL B STORAGE WERE NOT BEING STORED IN ACCORDANCE WITH APPROVED PROCEDURES. THE INSPECTOR DETERMINED THAT THE CONDITION HAD EXISTED FOR APPROXIMATELY FOUR AND ONE HALF WEEKS.
(7932 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

+ Unit operating routinely.

+LAST IE SITE INSPECTION DATE: June 15-17, 1980

+ INSPECTION REPORT NO: 80-09

SUMMARY

INSPECTION SUMMARY

+ Inspection on May 20-22, (80-08): Routine, unannounced inspection of: Technical Specification compliance during single loop operation, followup on outstanding inspection items, review of containment vent and purge line isolation valves. The inspection involved 17 inspector hours onsite by one NRC inspector. Of the areas inspected, no items of noncompliance were identified.

+ Inspection on June 16-17, (80-09): Routine, unannounced inspection of previously identified outstanding items and follow-up to IE Bulletin No. 79-02, Pipe Support Base Plate Designs Using Concrete Expansion Anchors. Review of licensee event reports on pipe supports; review of concrete cracks in floor under torus. The inspection involved 11 inspector hours onsite by one NRC inspector. No items of noncompliance were identified.

REPORTS RECEIVED FROM LICENSEE

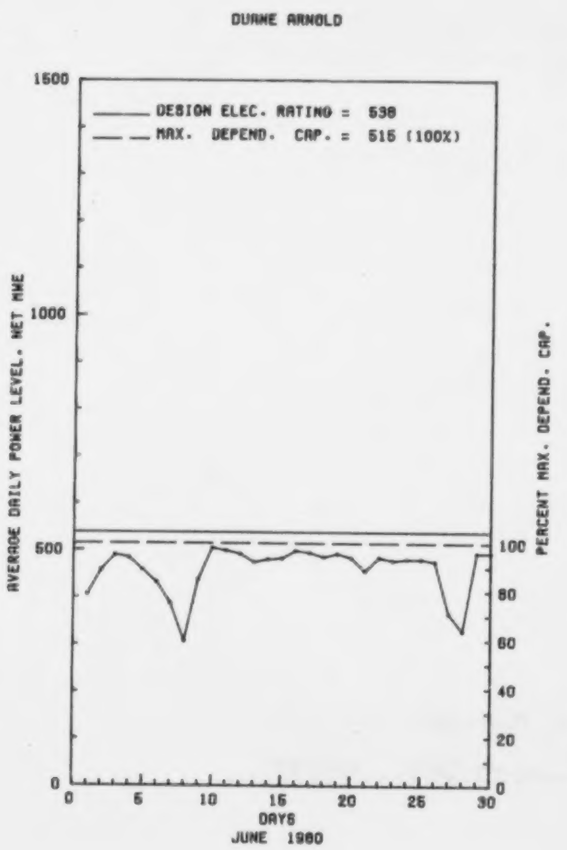
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-17/03L-0	04/30/80	05/27/80	The "A" reactor recirc. pump motor generator set tripped.
80-18/01T-0	05/19/80	05/27/80	During inspection of standby diesel gen. 1G-31, oil drain petcock on hydraulic gov. was open slightly and no oil level indicated on the sight glass.
80-19/01T-0	05/20/80	05/27/80	During shift alarm testing, 3-5AMP fuses which service the A, B, and C annunciator cabinets in panel 1C03 were blown.

REVIEWED BY OMPA LOEB: John J. Rizzo DATE: 8-4-80

REVIEWED BY NRR DL: Kenneth T. Eccleston DATE: 8-6-80

DUANE ARNOLD

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-331
2. Utility Contact: J. Van Sickel (319) 851-5611
3. Licensed Thermal Power (MWt): 1658
4. Nameplate Rating (Gross MWe): 663 x 0.9 = 597
5. Design Electrical Rating (Net MWe): 538
6. Maximum Dependable Capacity (Gross MWe): 545
7. Maximum Dependable Capacity (Net MWe): 515
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	4,367.0	67,447.0
12. Hours Reactor Was Critical	720.0	2,568.7	33,'32.8
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	720.0	2,486.4	32,305.5
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,056,336	3,097,512	40,727,072
17. Gross Electrical Energy (MWH)	351,949	1,048,275	13,472,641
18. Net Electrical Energy (MWH)	330,970	980,643	12,595,707
19. Unit Service Factor	100.0	56.9	68.1
20. Unit Availability Factor	100.0	56.9	68.1
21. Unit Capacity Factor (MDC Net)	89.3	43.6	51.5
22. Unit Capacity Factor (DER Net)	85.4	41.7	49.3
23. Unit Forced Outage Rate	.0	6.3	19.9

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
9	06/08/80	S	.0	B	5		CH	HTEXCH	POWER REDUCED TO PERFORM MAINTENANCE ON THE "A" REACTOR FEED PUMP OIL COOLER.
10	.3/27/80	S	.0	A	5		CB		POWER WAS REDUCED TO REPAIR THE "B" RECIRCULATION SYSTEM M-G SET GENERATOR EXCITER.

SUMMARY

THE PLANT OPERATED NORMALLY FOR THE REPORTING PERIOD WITH BRIEF REDUCTIONS ON 06/08/80 AND 06/27/80 FOR MAINTENANCE TO FEED PUMP AND GENERATOR.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

FARLEY 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Alabama/Houston 24 miles SE of Dothan
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2652
5.	Design Electrical Rating (MWE Net)	829
6.	Date of Initial Criticality	August 9, 1977
7.	Date of Commercial Operation	December 1, 1977
8.	Condenser Cooling Method	Once Through With Forced Draft Cooling Towers
9.	Condenser Cooling Water Source	Chattahoochee River

Utility & Contractor Information

10.	Licensee	Alabama Power Company
11.	Corporate Address	600 North 18th Street Birmingham, Alabama
12.	Corporate Contact	F. L. Clayton, Jr. Senior Vice President (205-323-5341)
13.	Architect/Engineer	Bechtel/SSI
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Daniel

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. Lewis
18.	IE Section Chief	R. D. Martin
19.	IE Resident Inspector	W. H. Bradford (205-899-3386)
20.	Licensing Project Manager	E. A. Reeves
21.	Docket Number	50-348
22.	License Number/Date of Issue	NPF-2, 6/25/77
23.	Public Document Room	G. S. Houston Memorial Library 212 W. Burdeshaw Street Dothan, Alabama 36301

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

UNIT REMOVED FROM SERVICE FOR STEAM GENERATOR TUBE REPAIR.

+ LAST IE SITE INSPECTION DATE: JUNE 1-28, 1980

+ INSPECTION REPORT NO: 50-348/80-15

SUMMARY

INSPECTION SUMMARY

+ INSPECTION MAY 27 (80-11): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 6 INSPECTOR-HOURS ON SITE IN THE AREAS OF OBSERVING A MAJOR RADIATION EMERGENCY EXCERCISE. OF THE ONE AREA INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION APRIL 28 - MAY 30 (80-13): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 110 INSPECTOR-HOURS ON SITE IN THE AREAS OF UNIT NO. 2 HOT FUNCTIONAL TESTING, UNIT NO. 2 PREOPERATIONAL TESTING, UNIT 2 PLANT TESTING STATUS, UNIT 2 PLANT TOUR, UNIT 1 PLANT OPERATION, UNIT 2 STRUCTURAL INTEGRITY TEST, ESF SYSTEM VERIFICATIONS, AND SITE EMERGENCY DRILL. OF THE 8 AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

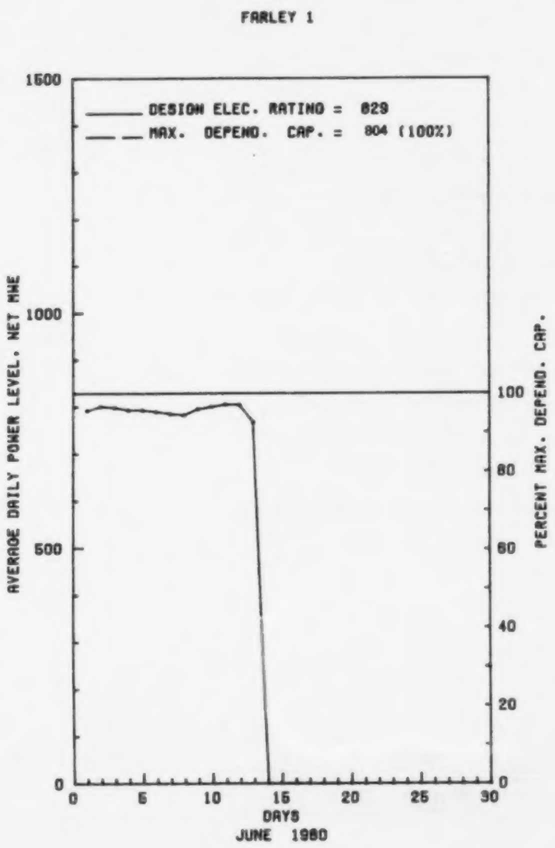
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-028/03L-0	04/24/80	05/19/80	DIESEL GENERATOR 1B INOPERABLE DUE TO BLOWN FUSES ON CONTROL POWER TO DIESEL'S VOLTAGE CONTROL CIRCUIT LICENSEE INVESTIGATING CAUSE OF FUSE BLOWING.
80-029/03L-0	04/30/80	05/28/80	LOSS OF POWER TO INTERMEDIATE RANGE NIS CHANNEL N-35.
80-030/03L-0	05/01/80	05/28/80	FUEL STORAGE POOL AREA RADIATION MONITOR R25A AND CONTAINMENT PURGE RADIATION MONITOR R24A INOPERABLE DUE TO BLOWN FUSE.
80-031/03L-0	05/11/80	06/10/80	BOTH TRAINS OF THE DIGITAL ROD POSITION INDICATION SYSTEM WERE LOST DUE TO CONVERSION CARD FAILURE.

REVIEWED BY OMPA LOEB: Philip A. Ross **DATE:** 08/04/80

REVIEWED BY NRR DL: R [illegible] **DATE:** 08/05/80

FARLEY 1

AVERAGE POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-348
2. Utility Contact: W. G. Hairston,III (205) 899-5156
3. Licensed Thermal Power (MWt): 2652
4. Nameplate Rating (Gross MWe): 1045 x 0.85 = 888
5. Design Electrical Rating (Net MWe): 829
6. Maximum Dependable Capacity (Gross MWe): 845
7. Maximum Dependable Capacity (Net MWe): 804
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

ITEM 21 CALCULATED WITH A WEIGHTED MDC

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	4,367.0	22,631.0
12. Hours Reactor Was Critical	314.5	3,525.1	14,460.6
13. Reactor Reserve Shutdown Hours	405.5	841.9	2,247.2
14. Hours Generator On-Line	313.5	3,461.2	14,051.7
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	822,411	8,781,343	35,379,358
17. Gross Electrical Energy (MWH)	260,728	2,823,092	11,347,130
18. Net Electrical Energy (MWH)	242,782	2,663,926	10,467,932
19. Unit Service Factor	43.5	79.3	62.1
20. Unit Availability Factor	43.5	79.3	62.1
21. Unit Capacity Factor (MDC Net)	40.7	75.8	58.7
22. Unit Capacity Factor (DER Net)	40.7	73.6	56.9
23. Unit Forced Outage Rate	.0	3.6	6.6

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
REFUEL OUTAGE: LATE SEPT. 1980: APPROX 8 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: 07/04/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
046	06/01/80	S	.0	B	5		HA	VALVEX	CONTINUATION OF POWER REDUCTION FROM 05/31/80 FOR MAIN TURBINE GOVERNOR VALVE TESTING.
047	06/14/80	S	406.5	B	1		CC	HTEXCH	THE UNIT WAS TAKEN OFF-LINE TO REPAIR "B" STEAM GENERATOR TUBE LEAKAGE.

SUMMARY

THE UNIT OPERATED NEAR FULL POWER UNTIL SHUTDOWN ON 06/14/80 FOR STEAM GENERATOR REPAIR.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

FITZPATRICK

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	New York/Oswego 8 miles NE of Oswego
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2436
5.	Design Electrical Rating (MWE Net)	821
6.	Date of Initial Criticality	November 17, 1974
7.	Date of Commercial Operation	July 28, 1975
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Lake Ontario

Utility & Contractor Information

10.	Licensee	Power Authority of the State of New York
11.	Corporate Address	10 Columbus Circle New York, New York 10019
12.	Corporate Contact	P. Lyon, Manager of Nuclear Operations
13.	Architect/Engineer	Stone & Webster
14.	Nuclear Steam Supply System	GE
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Section Chief	H. B. KISTER
19.	IE Resident Inspector	W. H. BAUNACK
20.	Licensing Project Manager	P. Polk (X7872)
21.	Docket Number	50-333
22.	License Number/Date of Issue	DPR-59, 10/17/74
23.	Public Document Room	State University College of Oswego Penfield Library - Documents Oswego, NY 13126 (315)341-2323

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO TS 6.8.A & ANSI HL8.7 PARA. 5.3.6, PROGRAM FOR ESTABLISHING & IMPLEMENTING CALIBRATION PROCEDURES NOT ADEQUATELY IMPLEMENTED IN THAT PROCEDURES FOR CALIBRATION OF RCIC FLOW INSTRUMENTS & CORE FLOW DIFFERENTIAL PRESSURE INSTRUMENTS WERE NOT FOLLOWED, RCIC FLOW SWITCH NOT CALIBRATED, RCIC FLOW INDICATOR NOT COVERED BY PROCEDURE, CONTROL ROD DRIVE ACCUMULATOR PRESSURE & LEVEL ALARMS NOT CALIBRATED & NO PROCEDURE ESTABLISHED FOR CONTAINMENT LEAK RATE MONITOR INSTRUMENT & PRESSURE INSTRUMENT USED FOR TESTING STANDBY LIQUID CONTROL SYSTEM RELIEF VALVES.
(8005 INFRACTION)

CONTRARY TO TS 6.8.A, PROC. NO. 4.1, 1.4 & 4.2, THERE WAS INADEQUATE REVIEW OF TEST RESULTS IN THAT OUT-OF-CALIBRATION & IMPROPERLY CALCULATED TEST DATA WAS ACCEPTED. COVER SHEETS OF INSTRUMENT SURVEILLANCE PROCEDURES DO NOT SPECIFY LIST OF EFFECT PAGES & LATEST REVISIONS FOR PROCEDURE SIGNOFF & DATA SHEETS & 2 INSTRUMENTS NOT LOGGED FOR TRACEABILITY TO TESTS FOR WHICH THEY WERE USED.
(8005 DEFICIENCY)

CONTRARY TO TS 6.8.A & PROCEDURE 1.4, 132 OF 161 INSTRUMENT MAINTENANCE PROCEDURES HAVE NOT BEEN REVIEWED OR REVISED WITHIN LAST 2 1/2 YEARS.
(8005 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS:

- Fuel handling and refueling operations have been completed. Torus modifications are essentially complete. Local leak rate testing and fire protection modifications are in progress. The refueling outage is expected to be completed by [illegible]
- LAST IE SITE INSPECTION DATE: 6/27/80
- INSPECTION REPORT NO: 50-333/80-11

SUMMARY

INSPECTION SUMMARY

- 50-333/80-08 - May 19-23: Routine, unannounced inspection by three regional based inspectors (96 hours) of new fuel receipt, handling and inspection; refueling activities in the control room and on the fuel floor; surveillance testing related to refueling Technical Specifications; and, major maintenance work on MSIV's, emergency diesel generator, and removal of the reactor vessel head. No items of noncompliance were identified.
- 50-333/80-10 - May 14: Routine, announced inspection by one regional inspector (8 hours) of licensee RHR system components as identified in the May 1980 document entitled "Response to NRC IE Bulletin 79-01B for James A. FitzPatrick Nuclear Power Plant." No items of noncompliance were identified.

REPORTS RECEIVED FROM LICENSEE

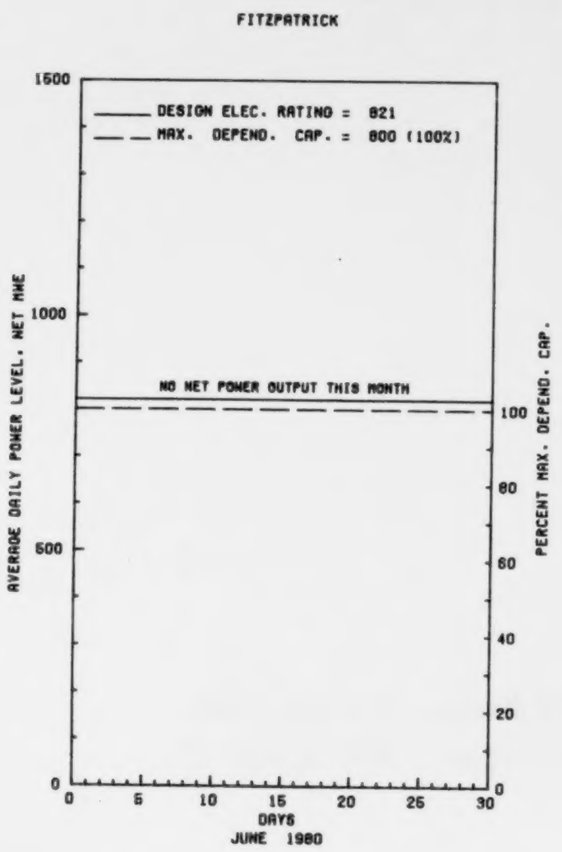
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-035/03L	04/15/80	05/15/80	Containment isolation valve failed in closed position.
80-036/03L	04/17/80	05/16/80	Insufficient inputs to one APRM Channel.
80-037/03L	04/18/80	05/16/80	Failure of one reactor water level transmitter.
80-038/03L	04/18/80	05/19/80	Containment isolation valve failure.
80-039/03L	04/21/80	05/19/80	Jet pump delta pressure transmitter instrument drift.
80-040/03L	04/22/80	05/21/80	LPCI MOV independent power supply inverter failure.
80-041/04T	05/13/80	05/27/80	Reactor building airborne release in excess of Technical Specification limit.
80-042/03L	04/28/80	05/27/80	SRM B and D recorder inoperable.
80-043/03L	04/28/80	05/27/80	APRM B scram and rod block setpoint 1 percent above limit.
80-044/03L	04/28/80	05/27/80	Off-gas timer setpoint error identified during surveillance testing.
80-045/03L	04/29/80	05/27/80	Rod block monitor A inoperable.
80-046/03L	04/30/80	05/27/80	Shutdown cooling permissive switch drift identified during surveillance testing.
80-047/03L	05/01/80	05/27/80	RCIC high ambient temperature switch inoperable.
80-048/03L	05/07/80	06/06/80	Discharge of laundry drain tank without obtaining required permit.
80-049/03L	05/07/80	06/06/80	Torus water level outside Technical Specification limit.
80-050/03L	05/09/80	06/06/80	Excessive main steam isolation valve leakage identified during testing.
80-051/03L	05/21/80	06/13/80	SRM recorder drift identified during surveillance testing.

REVIEWED BY OMPA LOEB: John J. Rizzo DATE: 8-4-80

REVIEWED BY NRR DL: Philip J. Polk DATE: 8/5/80

FITZPATRICK

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-333
2. Utility Contact: E. Zufelt (315) 342-3840
3. Licensed Thermal Power (MWt): 2436
4. Nameplate Rating (Gross MWe): 981 x 0.9 = 883
5. Design Electrical Rating (Net MWe): 821
6. Maximum Dependable Capacity (Gross MWe): 830
7. Maximum Dependable Capacity (Net MWe): 800
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

ITEM 21 CALCULATED WITH A WEIGHTED MDC

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	4,367.0	43,230.0
12. Hours Reactor Was Critical	.0	2,936.1	29,566.3
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	.0	2,893.0	28,631.0
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	5,862,024	57,050,218
17. Gross Electrical Energy (MWH)	0	2,019,020	19,372,720
18. Net Electrical Energy (MWH)	0	1,953,370	18,743,965
19. Unit Service Factor	.0	66.2	66.3
20. Unit Availability Factor	.0	66.2	66.3
21. Unit Capacity Factor (MDC Net)	.0	55.9	58.3
22. Unit Capacity Factor (DER Net)	.0	54.5	52.8
23. Unit Forced Outage Rate	.0	5.4	19.7

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 08/08/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
14	05/07/80	S	720.0	C	4		RC	FUELXX	REFUELING OUTAGE (CUMULATIVE HOURS TO DATE 1308).

SUMMARY

THIRD REFUELING AND MAINTENANCE OUTAGE CONTINUED THROUGH THE REPORTING PERIOD.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

FORT CALHOUN

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Nebraska/Washington 19 miles N of Omaha
2.	Electric Reliability Council	Mid-Continent Area Reliability Coordination Agreement
3.	Type of Reactor	PWR
4.	Power Level (MWT)	1420
5.	Design Electrical Rating (MWE Net)	457
6.	Date of Initial Criticality	August 6, 1973
7.	Date of Commercial Operation	June 20, 1974
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Missouri River

Utility & Contractor Information

10.	Licensee	W. C. Jones Division Manager – Production Operations Omaha Public Power District
11.	Corporate Address	1623 Harney Street Omaha, Nebraska 68102
12.	Corporate Contact	T. E. Short Assistant General Manager
13.	Architect/Engineer	Gibbs & Hill, Inc.
14.	Nuclear Steam Supply System	Combustion Engineering
15.	Constructor	Gibbs & Hill, Durham & Richardson, Inc.

Regulatory Information

16.	IE Region responsible	IV
17.	IE Branch Chief	G. L. Madsen
18.	IE Section Chief	D. M. Hunnicutt
19.	IE Resident Inspector	D. Kelley
20.	Licensing Project Manager	P. C. Wagner
21.	Docket Number	50-285
22.	License Number/Date of Issue	DPR-40, 5/24/73
23.	Public Document Room	W. Dale Clark Library 215 S. 15th Street Omaha, Nebraska 68102

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

FACILITY ITEMS (PLANS AND PROCEDURES):

A RADIATION SURVEY REQUIRED BY TECHNICAL SPECIFICATION 5.8.1 FOR ROOM 15A HAD NOT BEEN CONDUCTED FOR APRIL 1980, WHICH IS CONTRARY TO THE LICENSEE'S PROCEDURES.

MANAGERIAL ITEMS:

PLANT STATUS:

No significant information

LAST IE SITE INSPECTION DATE: MAY 14 - 16, 1980

INSPECTION REPORT NO: 50-285/80-10

SUMMARY

INSPECTION SUMMARY

INVESTIGATION ON MAY 14-16, 1980 (80-10): ALLEGATIONS THAT THE UTILITY KNOWINGLY PERMITTED WORKERS TO WORK IN AN AREA WHERE RADIATION EXPOSURES WERE GREATER THAN THE WORKERS WERE LED TO BELIEVE. THE ALLEGATION WAS NOT SUBSTANTIATED; HOWEVER, DURING THE INVESTIGATION ONE ITEM OF NON-COMPLIANCE NOT DIRECTLY ASSOCIATED WITH THE ALLEGATIONS WAS IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

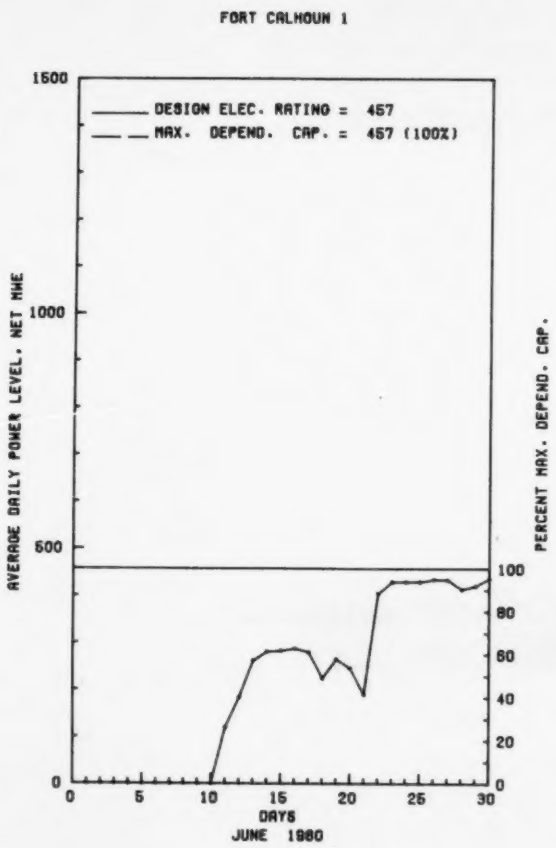
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-11 03L-0	4/16/80	5/12/80	EXCESS CONTAINMENT LEAKAGE FOR TYPE B AND C CONTAINMENT LEAK RATE TESTS IDENTIFIED DURING ILRT FOR 1980

REVIEWED BY OMPA LOEB: MR Beebe **DATE:** 08/04/80

REVIEWED BY NRR DL: PC Wagner **DATE:** 8/5/80

FORT CALHOUN

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-285
2. Utility Contact: B. J. Hickle (402) 536-4413
3. Licensed Thermal Power (MWt): 1420
4. Nameplate Rating (Gross MWe): 591 x 0.85 = 502
5. Design Electrical Rating (Net MWe): 457
6. Maximum Dependable Capacity (Gross MWe): 481
7. Maximum Dependable Capacity (Net MWe): 457
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

ITEM 21 CALCULATED WITH A WEIGHTED MDC.

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	4,367.0	59,304.0
12. Hours Reactor Was Critical	534.9	967.8	45,369.4
13. Reactor Reserve Shutdown Hours	.0	.0	1,309.5
14. Hours Generator On-Line	478.5	909.0	44,360.1
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	512,426	1,087,157	54,322,104
17. Gross Electrical Energy (MWH)	165,142	355,746	18,015,976
18. Net Electrical Energy (MWH)	154,751	335,686	17,023,203
19. Unit Service Factor	66.5	20.8	74.8
20. Unit Availability Factor	66.5	20.8	74.8
21. Unit Capacity Factor (MDC Net)	47.0	16.8	63.2
22. Unit Capacity Factor (DER Net)	47.0	16.8	62.8
23. Unit Forced Outage Rate	.0	.0	4.4

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-01	01/18/80	S	241.5	C	4		RC	FUELXX	REACTOR SHUTDOWN FOR ANNUAL REFUELING AND MAINTENANCE OUTAGE (3458.0 HRS TOTAL).

SUMMARY

THE UNIT CAME ON LINE ON 06/11/80 FOLLOWING A REFUELING OUTAGE. IT REACHED NORMAL POWER PRODUCTION ON 06/22/80 AND RAN NORMALLY FOR REST OF THE MONTH.

LEGEND

[1] Type:
- F–Forced
- S–Scheduled

[2] Reason:
- A–Equipment Failure
- B–Maintenance or Test
- C–Refueling
- D–Regulatory Restriction
- E–Operator Training & License Examination
- F–Administrative
- G–Operational Error
- H–Other

[3] Method:
- 1–Manual
- 2–Manual Scram
- 3–Automatic Scram
- 4–Continuations
- 5–Load Reduction
- 9–Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

FORT ST. VRAIN

NRC POWER RESTRICTIONS: Restricted to 70% of Licensed Thermal Power.

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Colorado/Weld 35 miles N of Denver (near Platteville)
2.	Electric Reliability Council	Western Systems Coordinating Council
3.	Type of Reactor	HTGR
4.	Power Level (MWT)	842
5.	Design Electrical Rating (MWE Net)	330
6.	Date of Initial Criticality	January 31, 1974
7.	Date of Commercial Operation	July 1, 1979
8.	Condenser Cooling Method	Towers
9.	Condenser Cooling Water Source	South Platte River/Shallow Wells

Utility & Contractor Information

10.	Licensee	Public Service Company of Colorado
11.	Corporate Address	P. O. Box 840 Denver, Colorado 80201
12.	Corporate Contact	C. K. Millen Senior Vice President
13.	Architect/Engineer	Sargent & Lundy
14.	Nuclear Steam Supply System	General Atomic Company
15.	Constructor	General Atomic Company

Regulatory Information

16.	IE Region responsible	IV
17.	IE Branch Chief	G. L. Madsen
18.	IE Section Chief	T. F. Westerman
19.	IE Resident Inspector	M. W. Dickerson (303) 785-2281
20.	Licensing Project Manager	G. Kuzmycz
21.	Docket Number	50-267
22.	License Number/Date of Issue	DPR-34, 12/21/73
23.	Public Document Room	Greeley Public Library City Complex Building Greeley, Colorado 80631

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO TECH SPEC 7.4 WHICH REQUIRES THAT WRITTEN PROCEDURES BE ESTABLISHED, IMPLEMENTED, AND MAINTAINED AND ADM-29, ADMINISTRATIVE PROCEDURE FOR CONTROL OF IGNITION SOURCES DURING MAINTENANCE, A FIRE WATCH WAS NOT PERFORMING HIS SOLE DUTY OF WATCHING FOR SPARKS DURING WELDING OPERATIONS AND A FIRE EXTINGUISHER WAS NOT "AT THE SITE" OF WELDING OPERATIONS.
(8010 INFRACTION)

TECH SPEC 7.4 REQUIRES THAT CHANGES TO PROCEDURES BE REVIEWED AND APPROVED. CONTRARY TO THIS REQUIREMENT, SURVEILLANCE PROCEDURES SR5.1.3-R AND SK5.2.20B-M WERE CHANGED AND THE NECESSARY REVIEWS AND APPROVALS WERE NOT DONE.
(8010 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

Areas related to potential equipment modification, which are under review and discussion with the NRC include: helium circulator auxiliaries; accident analysis relating to outlet temperature dispersion and orifice settings; hangers for 2" and smaller piping; and plant oscillations.

FACILITY ITEMS (PLANS AND PROCEDURES):

IMPLEMENTATION OF 10 CFR 73.55 PROVISIONS WERE IN EFFECT 3/21/80.

MANAGERIAL ITEMS:

PLANT STATUS:

PLANT IS OPERATING AT 53% POWER. +

LAST IE SITE INSPECTION DATE: MAY 1 - 31, 1980 +

INSPECTION REPORT NO: 50-267/80-10 +

SUMMARY

INSPECTION SUMMARY

+ INSPECTION ON MAY 27-28, 1980 (80-11): ROUTINE, UNANNOUNCED INSPECTION RELATED TO THE LICENSEE'S PROGRAM AND ACTIVITIES DESIGNED TO PROTECT AGAINST INDUSTRIAL SABOTAGE AND TO SAFEGUARD SPECIAL NUCLEAR MATERIAL. NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION ON MAY 1-31, 1980 (80-10): ROUTINE, ANNOUNCED INSPECTION OF REVIEW OF SURVEILLANCE TESTING AND CALIBRATION CONTROL PROGRAM; REVIEW OF PLANT OPERATIONS; REVIEW OF EVENT REPORTS; FOLLOW-UP ON INSPECTOR IDENTIFIED AND UNRESOLVED ITEMS; SURVEILLANCE; MAINTENANCE; OPERATIONAL SAFETY VERIFICATION; AND REVIEW OF BULLETINS. WITHIN THE EIGHT (8) AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN SIX AREAS. TWO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED (INFRACTION - FAILURE TO MEET FIRE WATCH AND EXTINGUISHER REQUIREMENTS, AND INFRACTION - FAILURE TO OBTAIN APPROVAL FOR CHANGES MADE TO PROCEDURES, IN TWO AREAS.

REPORTS RECEIVED FROM LICENSEE

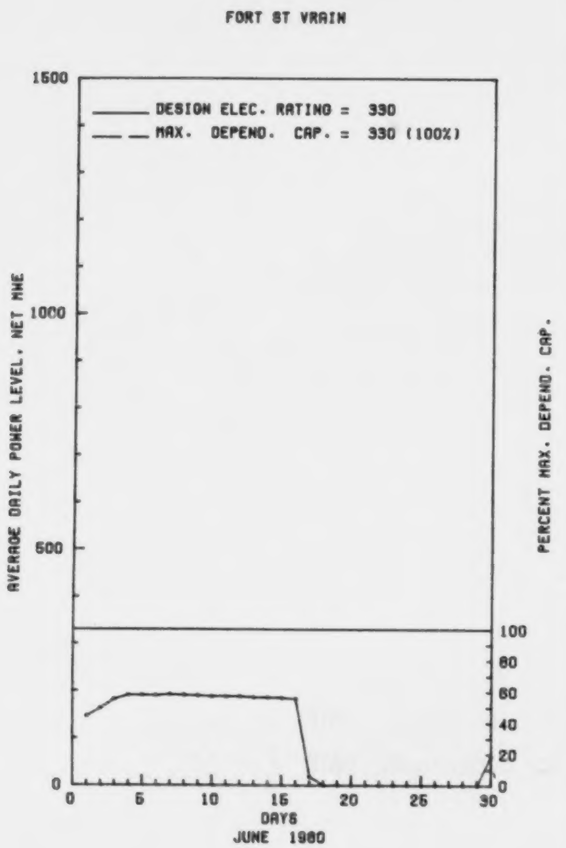
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-23 03L-0	5/7/80	6/6/80	PLANT OPERATION IN A DEGRADED MODE IN THAT THE EMERGENCY FEEDWATER SUPPLY TO ONE HELIUM CIRCULATOR WATER TURBINE WAS ISOLATED FOR MAINTENANCE ON FOUR SEPARATE OCCASIONS
80-24 03L-0	5/12/80	6/6/80	HELIUM DIFFERENTIAL PRESSURE INSTRUMENT EQUALIZATION VALVE ON THE 1C CIRCULATOR BUFFER FOUND OPEN
80-25 03L-0	5/23/80	6/19/80	TOTAL PRIMARY COOLANT OXIDANTS GREATER THAN TECHNICAL SPECIFICATION LIMITS
80-26 03L-0	5/24/80	6/19/80	EIGHT OF TWELVE HELIUM CIRCULATOR SEAL MALFUNCTION PRESSURE DIFFERENTIAL SWITCH UNITS WERE FOUND TO TRIP OUTSIDE THE TECHNICAL SPECIFICATION LIMITS

REVIEWED BY OMPA LOEB: Philip A. Ross **DATE:** 08/04/80

REVIEWED BY NRR DL: James R. Miller **DATE:** 8/6/80

FORT ST. VRAIN

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-267
2. Utility Contact: J. W. GAHM (303) 785-2253
3. Licensed Thermal Power (MWt): 842
4. Nameplate Rating (Gross MWe): 403 x 0.85 = 343
5. Design Electrical Rating (Net MWe): 330
6. Maximum Dependable Capacity (Gross MWe): 342
7. Maximum Dependable Capacity (Net MWe): 330
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): 231
10. Reasons for Restrictions, If Any: RESTRICT. PEND. RESOL. OF TEMP. FLUCTUATIONS

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	4,367.0	8,784.0
12. Hours Reactor Was Critical	720.0	3,824.9	5,557.7
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	406.6	2,053.5	3,035.7
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	244,228	969,290	1,447,245
17. Gross Electrical Energy (MWH)	75,923	312,946	453,742
18. Net Electrical Energy (MWH)	70,314	290,327	413,911
19. Unit Service Factor	56.5	47.0	34.6
20. Unit Availability Factor	56.5	47.0	34.6
21. Unit Capacity Factor (MDC Net)	29.6	20.1	14.3
22. Unit Capacity Factor (DER Net)	29.6	20.1	14.3
23. Unit Forced Outage Rate	43.5	27.3	46.2

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): AUG. 30 - SEPT. 30, COMPLETE REQUIRED SURVEILLANCE TESTING.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-12	06/17/80	F	313.4	H	9				TURBINE MANUALLY SHUTDOWN AFTER BOTH CIRCULATORS ON ONE LOOP TRIPPED. LOOP ISOLATED, REACTOR NOT SHUTDOWN.

SUMMARY

THE UNIT OPERATED ROUTINELY AT REDUCED POWER UNTIL SHUTDOWN ON 06/17/80. THE PLANT RETURNED TO SERVICE ON 06/30/80.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

GINNA

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	New York/Wayne 15 miles NE of Rochester
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	1520
5.	Design Electrical Rating (MWE Net)	470
6.	Date of Initial Criticality	November 8, 1969
7.	Date of Commercial Operation	July 1970
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Lake Ontario

Utility & Contractor Information

10.	Licensee	Rochester Gas & Electric Corporation
11.	Corporate Address	89 East Avenue Rochester, New York
12.	Corporate Contact	Leon D. White, Jr., Vice President Electric and Steam Production
13.	Architect/Engineer	Gilbert
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Section Chief	H. B. KISTER
19.	IE Resident Inspector	D. JOHNSON
20.	Licensing Project Manager	S. J. Nowicki
21.	Docket Number	50-244
22.	License Number/Date of Issue	DPR-18, 9/19/69
23.	Public Document Room	Rochester Public Library Business and Social Science Division 115 South Avenue Rochester, New York 14604

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS:

+ Operating at 100% power.

+ LAST IE SITE INSPECTION DATE: 6/13/80

+ INSPECTION REPORT NO: 50-244/80-09

SUMMARY

INSPECTION SUMMARY

+ None.

REPORTS RECEIVED FROM LICENSEE

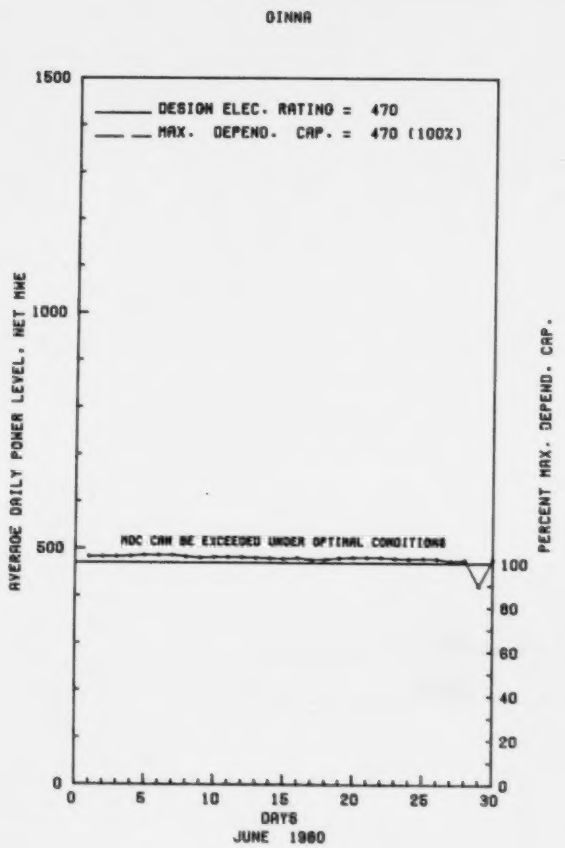
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-005/01T	06/03/80	06/16/80	During performance of surveillance proc for verification of safeguards equipment the breaker for A loop accumulator disc valve was found in closed position contrary to TS 3.3.1.1G.

REVIEWED BY OMPA LOEB: Leonard Toman **DATE:** 08/04/80

REVIEWED BY NRR DL: S. J. Nowicki **DATE:** 8/5/80

GINNA

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-244
2. Utility Contact: A.E. MCNAMARA (315) 524-4446
3. Licensed Thermal Power (MWt): 1520
4. Nameplate Rating (Gross MWe): 608 x 0.85 = 517
5. Design Electrical Rating (Net MWe): 470
6. Maximum Dependable Capacity (Gross MWe): 490
7. Maximum Dependable Capacity (Net MWe): 470

CAPACITY FACTORS CALCULATED WITH A WEIGHTED AVERAGE

8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	4,367.0	92,879.0
12. Hours Reactor Was Critical	720.0	3,066.0	70,745.8
13. Reactor Reserve Shutdown Hours	.0	.0	1,621.8
14. Hours Generator On-Line	720.0	3,027.3	68,984.1
15. Unit Reserve Shutdown Hours	.0	.0	8.5
16. Gross Thermal Energy (MWH)	1,084,200	4,441,800	92,717,602
17. Gross Electrical Energy (MWH)	362,152	1,460,888	30,146,679
18. Net Electrical Energy (MWH)	344,844	1,388,807	28,550,355
19. Unit Service Factor	100.0	69.3	74.3
20. Unit Availability Factor	100.0	69.3	74.3
21. Unit Capacity Factor (MDC Net)	101.9	67.7	67.7
22. Unit Capacity Factor (DER Net)	101.9	67.7	67.7
23. Unit Forced Outage Rate	.0	.0	9.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
*SCHED. - STEAM GEN. INSPECTION & MOD - 10/24 THRU 11/23/80.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence

SUMMARY

THE UNIT RAN NORMALLY FOR REPORTING PERIOD WITH A BRIEF REDUCTION ON 06/29/80 DUE TO MAINTENANCE TESTS.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

HADDAM NECK

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Connecticut/Middlesex 13 miles E of Meriden
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	1825
5.	Design Electrical Rating (MWE Net)	575
6.	Date of Initial Criticality	July 24, 1967
7.	Date of Commercial Operation	January 1, 1968
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Connecticut River

Utility & Contractor Information

10.	Licensee	Connecticut Yankee Atomic Power Co.
11.	Corporate Address	P. O. Box 270 Hartford, Connecticut
12.	Corporate Contact	D. Switzer, President
13.	Architect/Engineer	Stone & Webster
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	I
17.	IE Section Chief	E. J. Brunner
18.	IE Resident Inspector	R. Keimig
19.	IE Principal Inspector	T. H. Smith
20.	Licensing Project Manager	R. Caruso
21.	Docket Number	50-213
22.	License Number/Date of Issue	DPR-61, 12/27/74
23.	Public Document Room	Russell Library 119 Broad Street Middletown, Connecticut 06457

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO TS 2.4.3, "SPECIFICATION FOR GASEOUS WASTE EFFLUENTS," ON 5/4/80, AT ABOUT 7:55 A.M. & 9:00 A.M., THE RELEASE RATE LIMIT OF NOBLE GASES EXCEEDED THE TS LIMIT BY A FACTOR OF 1.66 & 2.78, RESPECTIVELY, FOR A PERIOD OF 4 MINUTES EACH.
(8809 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

- FACILITY ITEMS (PLANS AND PROCEDURES):
- The facility is currently in a normal refueling outage which is scheduled to be completed on July 13, 1980.

PLANT STATUS:

- Shutdown for refueling since May 3, 1980.
- LAST IE SITE INSPECTION DATE: 6/23/80
- INSPECTION REPORT NO: 50-213/80-12

SUMMARY

INSPECTION SUMMARY

- 50-213/80-04 - Mar 18-21: Routine, unannounced inspection by one regional based inspector (20 hours) of emergency procedure conformance to Westinghouse Small Break Loss of Coolant Operator Guidelines and status of licensee investigation concerning contamination discovered on plant property outside the radiologically controlled area. No items of noncompliance were identified.
- 50-213/80-08 - Apr 21-25: Routine, unannounced inspection by two regional based inspectors (59 hours) on previous inspection findings; administrative controls for facility procedures; conformance to Technical Specifications; temporary and permanent changes in conformance to Technical Specifications and licensee procedures; changes in procedures to 10 CFR 50.59(a) and (b) requirements; checklists and related forms for currency to latest changes; administrative controls for safety-related maintenance; safety-related maintenance activities; maintenance personnel qualifications; and control room and facility tours. Five items of noncompliance were identified: failure to properly approve procedures and failure to conduct PORC meetings with a quorum; failure to establish written procedures for annunciators modifications; failure to use procedures and to perform post maintenance inspections and tests for safety-related maintenance; failure to use latest revision of a procedure and failure to post procedure revisions; and, failure to maintain records of maintenance activities.
- 50-213/80-09 - May 4-5: Special, announced inspection by one regional based inspecter (8 hours) to follow up site noble gas releases, which occurred on May 4, 1980, specifically in the areas of notifications; Emergency Plan implementation; and Technical Specification release rate limits. One item of noncompliance: failure to adhere to Technical Specification limits regarding noble gas release rates.
- 50-213/80-10 - May 19-23: Physical Security Inspection by one regional based inspector (34 hours) of: Security Plan; Security Organization; Physical Barriers (Protected Area); Physical Barriers (Vital Area); Lighting; Access Control (Identification, Authorization, Badging); Access Control (Search); Access Control (Escorting); Access Control (Vital Area); Detection Aids; Alarm Stations; Assessment Aids; Communications; Testing and Maintenance; Compensatory Measures; Power Supply; Response; Locks, Keys and Combinations; and Records and Reports. Also a followup on previously identified items of noncompliance. No items of noncompliance were identified.

REPORTS RECEIVED FROM LICENSEE

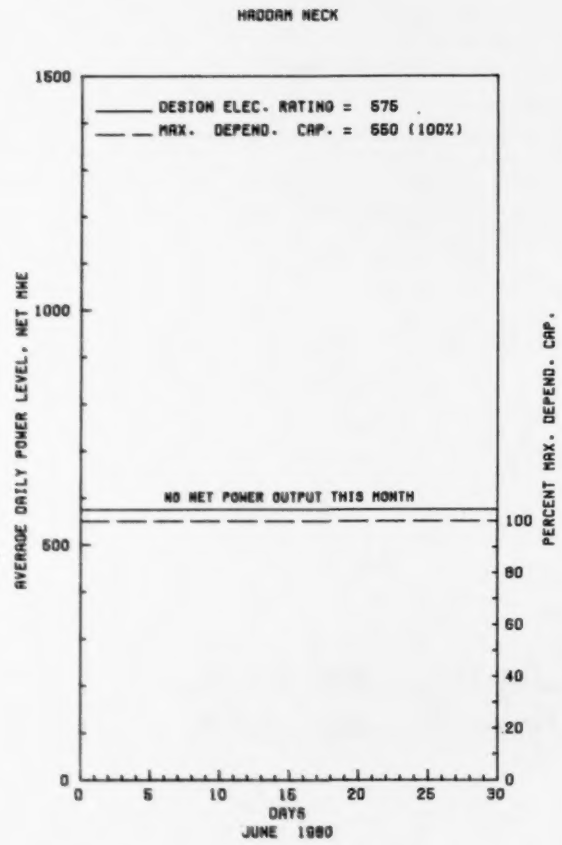
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-009/01T	05/16/80	05/30/80	Acceptance criteria exceeded during performance of local leak rate testing of penetrations

REVIEWED BY OMPA LOEB: Leonard J. Tomas **DATE:** 08/04/80

REVIEWED BY NRR DL: __________ **DATE:** __________

HADDAM NECK

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-213
2. Utility Contact: D. ANDERSON (203) 267-2556 X274
3. Licensed Thermal Power (MWt): 1825
4. Nameplate Rating (Gross MWe): 667 x 0.9 = 600
5. Design Electrical Rating (Net MWe): 575
6. Maximum Dependable Capacity (Gross MWe): 577
7. Maximum Dependable Capacity (Net MWe): 550
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

CAPACITY FACTORS CALCULATED WITH A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	4,367.0	109,559.0
12. Hours Reactor Was Critical	.0	2,957.1	94,153.8
13. Reactor Reserve Shutdown Hours	.0	11.6	1,166.0
14. Hours Generator On-Line	.0	2,947.7	89,387.9
15. Unit Reserve Shutdown Hours	.0	.0	369.9
16. Gross Thermal Energy (MWH)	0	5,090,070	155,231,219
17. Gross Electrical Energy (MWH)	0	1,692,421	51,046,887
18. Net Electrical Energy (MWH)	-1,968	1,609,319	48,564,082
19. Unit Service Factor	.0	67.5	82.0
20. Unit Availability Factor	.0	67.5	82.4
21. Unit Capacity Factor (MDC Net)	.0	67.0	81.9
22. Unit Capacity Factor (DER Net)	.0	64.1	75.6
23. Unit Forced Outage Rate	.0	.5	7.1

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
CONTIN OF CORE 10 REFUEL. MAY 3, 1980, APPROX. 8 TO 12 WKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: 07/14/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-3	05/03/80	S	720.0	C	4		RC	FUELXX	CONTINUATION OF CORE 9 TO CORE 10 REFUELING.

SUMMARY

CONTINUATION OF REFUELING AND MAINTENANCE OUTAGE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

NRC POWER RESTRICTIONS: NONE

HATCH 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Georgia/Appling 11 miles N of Baxley
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2436
5.	Design Electrical Rating (MWE Net)	777
6.	Date of Initial Criticality	September 12, 1974
7.	Date of Commercial Operation	December 31, 1975
8.	Condenser Cooling Method	Once Through With Forced Draft Cooling Towers
9.	Condenser Cooling Water Source	Altamaha River

Utility & Contractor Information

10.	Licensee	Georgia Power Company
11.	Corporate Address	270 Peachtree Street, NE Atlanta, Georgia 30303
12.	Corporate Contact	J. H. Miller Executive Vice President
13.	Architect/Engineer	Southern Services, Inc.
14.	Nuclear Steam Supply System	GE
15.	Constructor	Georgia Power Company

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. Lewis
18.	IE Section Chief	H. C. Dance
19.	IE Resident Inspector	R. F. Rogers (912) 367-9881
20.	Licensing Project Manager	D. Verrelli
21.	Docket Number	50-321
22.	License Number/Date of Issue	DPR-57, 8/6/74
23.	Public Document Room	Appling County Public Library Parker Street Baxley, Georgia 31513

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO 10CFR71.5 THE LICENSEE DID NOT COMPLY WITH DOT REGULATIONS DURING A SHIPMENT OF RADIOACTIVE MATERIAL. THE DOSE RATE IN THE CAB OF THE VEHICLE WAS MEASURED TO BE 2.6-3.7 MREM/HR. 49CFR173.393(J)(4) SPECIFIES A MAXIMUM DOSE RATE OF 2.0 MREM/HR.
(8014 VIOLATION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

PLANT SHUTDOWN AT END OF MONTH DUE TO HPCI PROBLEMS.

◆ LAST IE SITE INSPECTION DATE: JUNE 5-6, 1980

◆ INSPECTION REPORT NO: 50-321/80-23

SUMMARY

INSPECTION SUMMARY

◆ INSPECTION MARCH 19 AND APRIL 29-30 (80-14): THIS WAS A SPECIAL INSPECTION OF A RADIOACTIVE WASTE SHIPMENT FROM PLANT HATCH TO THE CHEM-NUCLEAR DISPOSAL SITE NEAR BARNWELL, SOUTH CAROLINA. THE INSPECTION WAS CONDUCTED ON MARCH 19, 1980 AT THE DISPOSAL SITE AND ON APRIL 29 AND 30, 1980 AT THE HATCH SITE NEAR BAXLEY, GEORGIA. A ROUTINE INSPECTION WAS CONDUCTED ON MARCH 19, 1980 OF LICENSEE-SHIPPERS FOR COMPLIANCE WITH DEPARTMENT OF TRANSPORTATION (DOT) AND NUCLEAR REGULATORY COMMISSION (NRC) REGULATIONS FOR SHIPPING PAPER REQUIREMENTS, DOT PLACARDING REQUIREMENTS, RADIATION LEVELS, REMOVABLE CONTAMINATION, MARKING AND LABELING OF PACKAGES AND PROHIBITED ARTICLES OR CONTENTS. A SPECIAL INSPECTION WAS CONDUCTED ON APRIL 28-30, 1980 AT THE HATCH SITE IN THE AREAS OF RADIOACTIVE WASTE SHIPPING PROCEDURES, SHIPPING DOCUMENTS, SHIPPING SURVEYS, TECHNICIAN QUALIFICATIONS AND TRAINING, AND PORTABLE INSTRUMENTATION CALIBRATION. DETAILS I ADDRESSES INSPECTION ACTIVITIES AT THE WASTE DISPOSAL SITE. DETAILS II ADDRESSES INSPECTION ACTIVITIES AS THE HATCH SITE.

INSPECTION JUNE 5-6 (80-23): THIS SPECIAL, ANNOUNCED INSPECTION INVOLVED 5 INSPECTOR-HOURS ON SITE IN THE AREA OF SAFETY-RELATED MECHANICAL SNUBBERS. OF THE AREA INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

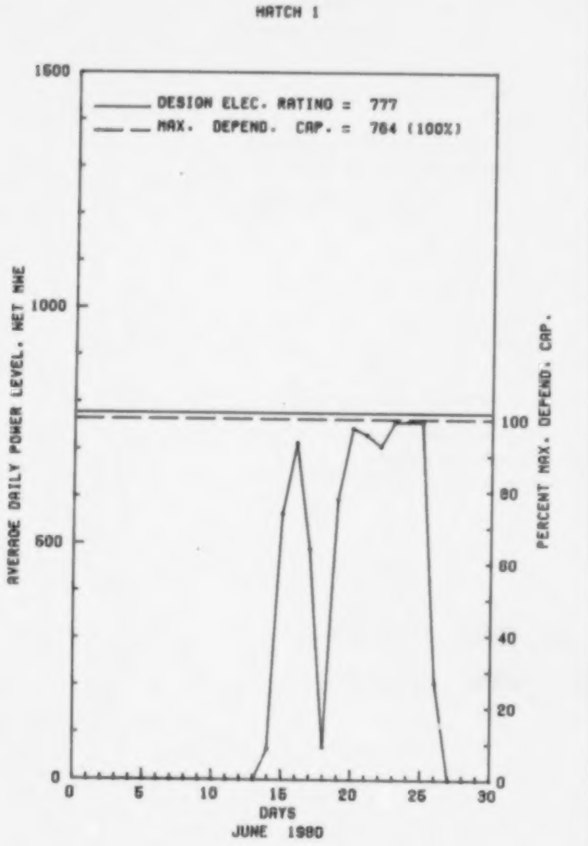
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-040	04/23/80	05/15/80	DRYWELL/TORUS DIFFERENTIAL PRESSURE RELIEF VALVE LIFTED.
80-041	04/25/80	05/07/80	LOW PRESSURE COOLANT INJECTION ISOLATION VALVES SEAT LEAKAGE.
80-042	04/29/80	05/20/80	RESIDUAL HEAT REMOVAL OUTBOARD CONTAINMENT SPRAY VALVE FAILED TO OPEN DURING TEST.
80-043	05/03/80	05/20/80	TORUS AIR AND WATER TEMPERATURE RECORDER INOPERABLE.
80-044	05/06/80	05/22/80	CONTAINMENT OXYGEN CONCENTRATION EXCEED ALLOWABLE.
80-045	05/10/80	05/22/80	COOLING WATER SUPPLY FOUND ISOLATED TO THE RESIDUAL HEAT REMOVAL SERVICE WATER PUMP MOTOR.
80-046/ 03L-0	04/27/80	05/20/80	DRYWELL/TORUS PRESSURE RECORDER INOPERABLE.
80-047	04/04/80	06/02/80	RESIDUAL HEAT REMOVAL PUMPS "A" AND "C" FLOW RATES DID NOT MEET REQUIREMENTS.
80-048	05/09/80	05/29/80	PLANT SERVICE WATER PUMP EXHIBITS LOW FLOW.
80-049	05/13/80	05/19/80	MAIN STEAM ISOLATION VALVES CLOSED INADVERTENTLY DURING TESTING.
80-050	05/14/80	06/03/80	REACTOR CORE ISOLATION COOLING TURBINE TRIPPED DURING A MANUAL START.
80-053	05/24/80	06/06/80	OPERATING WHERE SINGLE FAILURE CRITERIA EXITS.

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 8-4-80

REVIEWED BY NRR DL: **DATE:** 8-5-80

HATCH 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-321
2. Utility Contact: R.M. BLACKWELL (912) 367-7781
3. Licensed Thermal Power (MWt): 2436
4. Nameplate Rating (Gross MWe): 1000 x 0.85 = 850
5. Design Electrical Rating (Net MWe): 777
6. Maximum Dependable Capacity (Gross MWe): 797
7. Maximum Dependable Capacity (Net MWe): 764
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	4,367.0	39,431.0
12. Hours Reactor Was Critical	392.0	3,631.1	29,429.1
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	270.0	3,421.1	27,689.5
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	568,900	7,717,740	58,070,775
17. Gross Electrical Energy (MWH)	180,680	2,537,300	18,508,240
18. Net Electrical Energy (MWH)	167,205	2,438,498	17,900,431
19. Unit Service Factor	37.5	78.3	70.2
20. Unit Availability Factor	37.5	78.3	70.2
21. Unit Capacity Factor (MDC Net)	30.4	73.1	59.4
22. Unit Capacity Factor (DER Net)	29.9	71.9	58.4
23. Unit Forced Outage Rate	62.4	21.6	16.8

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 07/03/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
8019	05/24/80	F	168.0	A	4	1-80-53	CF	VALVEX	SHUTDOWN DUE TO INOP RHR VALVE E11-F017.
8020	06/08/80	F	150.0	A	3	1-80-53	HJ	SUPORT	UNIT REMAINED DOWN FOR MECHANICAL SNUBBER REPAIR.
8021	06/14/80	F	3.0	A	3		HC	XXXXXX	RX SCRAM - LOSS OF CONDENSER VACUUM DUE TO STEAM JET AIR EJECTER FAILURE.
8022	06/15/80	F	15.0	A	1		HG	DEMINX	CONDENSATE DEMINS PROBLEMS - LOW DP ON SUCTION VALVES.
8023	06/16/80	F	12.0	A	5		RB	CONROD	DROP LOAD DUE TO CONTROL ROD PATTERN ADJUSTMENT - LOAD DROP ST STARTUP RAMP.
80-24	06/17/80	F	50.0	A	5		HC	ZZZZZZ	CONSIDERABLE LOAD REDUCTION DUE TO LOSS OF CONDENSER VACUUM - SJAE PROPORTIONAL BANDS OFF.
80-25	06/21/80	S	1.0	B	1		HA	TURBIN	WEEKLY TURB TEST ON 06/21/80 - EXTENSIVE CHECK OF BYPASS VALVES.
80-26	06/26/80	F	113.0	A	3	1-80-69	CJ	VALVEX	HPCI OUT OF SERVICE - PROBS. WITH ELECTRONIC AND HYDRAULIC SYSTEMS.

SUMMARY

SEVERAL SCRAMS AND REDUCTIONS OCCURRED DURING THE MONTH, INCLUDING OUTAGES FOR RHR VALVE REPAIR, MECHANICAL SNUBBER REPAIR, SJAE FAILURE AND LOSS OF HPCI SYSTEM.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

HATCH 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Georgia/Appling 11 miles N of Baxley
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2436
5.	Design Electrical Rating (MWE Net)	784
6.	Date of Initial Criticality	July 4, 1978
7.	Date of Commercial Operation	September 5, 1979
8.	Condenser Cooling Method	Cooling Towers
9.	Condenser Cooling Water Source	Altamaha River

Utility & Contractor Information

10.	Licensee	Georgia Power Company
11.	Corporate Address	270 Peach Street, NE Atlanta, Georgia 30303
12.	Corporate Contact	J. H. Miller Executive Vice President
13.	Architect/Engineer	Southern Services, Inc./Bechtel
14.	Nuclear Steam Supply System	GE
15.	Constructor	Georgia Power Company

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. Lewis
18.	IE Section Chief	H. C. Dance
19.	IE Resident Inspector	R. F. Rogers (912) 367-9881
20.	Licensing Project Manager	D. Verrelli
21.	Docket Number	50-366
22.	License Number/Date of Issue	NPF-5/June 13, 1978
23.	Public Document Room	Appling County Public Library Parker Street Baxley, Georgia 31513

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO TECHNICAL SPECIFICATION 5.5.1 PROCEDURE HNP-1 7800 WAS NOT FOLLOWED IN PERFORMING AIR FLOW MEASUREMENTS FOR AIR SAMPLES AND PROCEDURE HNP-1 7803 WAS NOT FOLLOWED IN THE TREATMENT OF COLLECTED MILK SAMPLES.
(7819 INFRACTION)

CONTRARY TO 10CFR71.5 THE LICENSEE DID NOT COMPLY WITH DOT REGULATIONS DURING A SHIPMENT OF RADIOACTIVE MATERIAL. THE DOSE RATE IN THE CAB OF THE VEHICLE WAS MEASURED TO BE 2.6-3.7 MREM/HR. 49CFR173.393(J)(4) SPECIFIES A MAXIMUM DOSE RATE OF 2.0 MREM/HR.
(8014 VIOLATION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

PLANT IS OPERATING AT FULL POWER.

+ LAST IE SITE INSPECTION DATE: JUNE 5-6, 1980

+ INSPECTION REPORT NO: 50-366/80-23

SUMMARY

INSPECTION SUMMARY

+ INSPECTION MARCH 19 AND APRIL 29-30 (80-14): THIS WAS A SPECIAL INSPECTION OF A RADIOACTIVE WASTE SHIPMENT FROM PLANT HATCH TO THE CHEM-NUCLEAR DISPOSAL SITE NEAR BARNWELL, SOUTH CAROLINA. THE INSPECTION WAS CONDUCTED ON MARCH 19, 1980 AT THE DISPOSAL SITE AND ON APRIL 29 AND 30, 1980 AT THE HATCH SITE NEAR BAXLEY, GEORGIA. A ROUTINE INSPECTION WAS CONDUCTED ON MARCH 19, 1980 OF LICENSEE-SHIPPERS FOR COMPLIANCE WITH DEPARTMENT OF TRANSPORTATION (DOT) AND NUCLEAR REGULATORY COMMISSION (NRC) REGULATIONS FOR SHIPPING PAPER REQUIREMENTS, DOT PLACARDING REQUIREMENTS, RADIATION LEVELS, REMOVABLE CONTAMINATION, MARKING AND LABELING OF PACKAGES AND PROHIBITED ARTICLES OR CONTENTS. A SPECIAL INSPECTION WAS CONDUCTED ON APRIL 20-30, 1980 AT THE HATCH SITE IN THE AREAS OF RADIOACTIVE WASTE SHIPPING PROCEDURES, SHIPPING DOCUMENTS, SHIPPING SURVEYS, TECHNICIAN QUALIFICATIONS AND TRAINING, AND PORTABLE INSTRUMENTATION CALIBRATION. DETAILS I ADDRESSES INSPECTION ACTIVITIES AT THE WASTE DISPOSAL SITE. DETAILS II ADDRESSES INSPECTION ACTIVITIES AS THE HATCH SITE.

- INSPECTION JUNE 5-6 (80-23): THIS SPECIAL, ANNOUNCED INSPECTION INVOLVED 5 INSPECTOR-HOURS ON SITE IN THE AREA OF SAFETY-RELATED MECHANICAL SNUBBERS. OF THE AREA INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

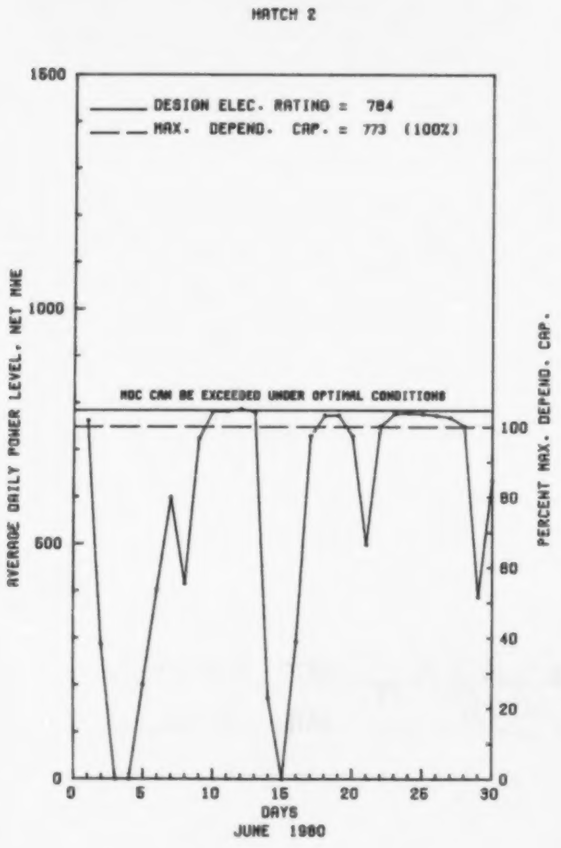
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-052/ 03L-0	04/08/80	05/05/80	POWER CABLES UNDERSIZE TO HEATERS AND FANS GAS TREATMENT SYSTEM.
80-053	05/05/80	05/15/80	HYDROGEN-OXYGEN ANALYZER PUMP DIAPHRAM SUBJECT TO RUPTURE IF DEADHEADED.
80-055	05/06/80	05/29/80	REACTOR CORE ISOLATION COOLING ISOLATION VALVES FAILED TO ISOLATE DURING TEST.
80-060/ 03L-0	04/11/80	05/07/80	"B" STANDBY GAS TREATMENT FILTER TRAIN INOPERABLE.
80-061/	04/17/80	05/13/80	CONTAINMENT AIRLOCK DOOR OPERABILITY TEST OF INTERLOCK NOT DONE.
80-062	04/19/80	05/06/80	DRYWELL TO TORUS VACUUM BREAKER INOPERABLE.
80-064	04/19/80	05/12/80	POSITION INDICATOR LIGHT FOR A MAIN STEAM ISOLATION VALVE FOUND INOPERABLE.
80-065	04/19/80	05/06/80	DRYWELL HYDROGEN-OXYGEN ANALYZER WILL BE DAMAGED WHEN ISOLATED AND VACUUM PUMP IN OPERATION.
80-066	04/22/80	05/07/80	RECEIVED SPURIOUS HIGH PRESSURE COOLANT INJECTION ISOLATION DURING TESTING.
80-067	04/22/80	05/07/80	"A" DRYWELL HYDROGEN-OXYGEN ANALYZER HAD DEFECTIVE CELL.
80-068	04/22/80	05/08/80	HYDROGEN-RECOMBINER FAILED TEMPERATURE TEST.
80-073	04/25/80	05/20/80	HYDROGEN-RECOMBINER FAILED TO START DURING TEST.
80-074	04/27/80	05/16/80	FAILED TO CALCULATE LIMITING POWER DENSITY AFTER DECREASE IN POWER.
80-075	04/28/80	05/16/80	TIME INTERVAL EXCEED FOR ALLOWED LIMITING POWER DENSITY.
80-076	04/29/80	05/23/80	CONTROL ROD DRIVE ACCUMULATOR PRESSURE SWITCHES SETPOINT DRIFT.
80-077	04/30/80	05/28/80	DRYWELL AVERAGE TEMPERATURE EXCEEDED DUE TO LOSS OF CHILLERS "A" AND "B".
80-078	04/19/80	05/12/80	RELAY CONTACTS FAILED TO OPEN DURING TESTING OF MAIN STEAM ISOLATION VALVE LEAKAGE CONTROL SYSTEM.
80-079	05/07/80	05/29/80	HIGH PRESSURE COOLANT INJECTION FAILS VALVE OPERABILITY TEST.
80-082	05/09/80	05/27/80	CONTAINMENT PURGE SYSTEM IN OPERATION LONGER THAN ALLOWED.
80-083	05/14/80	06/03/80	2D RESIDUAL HEAT REMOVAL PUMP SUCTION VALVES INOPERABLE.

REVIEWED BY OMPA LOEB: **DATE:** 8-4-80

REVIEWED BY NRR DL: **DATE:** 8-5-80

HATCH 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-366
2. Utility Contact: R.M. BLACKWELL (912) 367-7781
3. Licensed Thermal Power (MWt): 2436
4. Nameplate Rating (Gross MWe): 1000 x .85 = 850
5. Design Electrical Rating (Net MWe): 784
6. Maximum Dependable Capacity (Gross MWe): 806
7. Maximum Dependable Capacity (Net MWe): 773
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	4,367.0	7,200.0
12. Hours Reactor Was Critical	664.5	2,873.2	5,536.7
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	604.9	2,648.0	5,062.7
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,281,390	5,716,560	11,210,602
17. Gross Electrical Energy (MWH)	424,080	1,903,700	3,739,660
18. Net Electrical Energy (MWH)	404,108	1,811,241	3,548,378
19. Unit Service Factor	84.0	60.6	70.3
20. Unit Availability Factor	84.0	60.6	70.3
21. Unit Capacity Factor (MDC Net)	72.6	53.7	64.1
22. Unit Capacity Factor (DER Net)	71.6	52.9	63.2
23. Unit Forced Outage Rate	16.0	13.5	10.8

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-18	06/02/80	F	68.7	A	3		HA	TURBIN	RX SCRAM ON TURB. VIB. SNUBBER WORK PERFORMED. S/D EXTENDED FOR SNUBBER REPAIR IN CONDENSER BAY.
80-19	06/08/80	F	21.0	B	5		HC	HTEXCH	DROPPED TO CHECK FOR CONDENSER TUBE LEAKS.
80-20	06/14/80	F	46.4	A	3		HA	TURBIN	SCRAM ON TURB. VIB. MAIN BREARING #9.
80-21	06/20/80	F	18.0	A	5		HH	HTEXCH	REDUCED LOAD - REPAIR "B" 4TH STAGE HEATER.
80-22	06/28/80	S	28.0	B	5		RB	CONROD	REDUCED LOAD DURING ROD SEQUENCE EXCHANGE.

SUMMARY

THE UNIT OPERATED AT FULL POWER, EXCEPT FOR TWO SCRAMS. DUE TO TURBINE VIBRATIONS ON 06/02/80 SCRAM, MECHANICAL SNUBBERS WERE REPLACED; ON 06/14/80, THE TURBINE HIGH VIBRATION BEARING WAS REPLACED. THREE LOAD REDUCTIONS OCCURRED, FOR CONDENSER TUBE LEAK CHECKS, WEEKLY TURBINE TEST AND STAGE HEATER REPAIR, AND ROD SEQUENCE EXCHANGE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

HUMBOLDT BAY

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	California/Humboldt 4 miles S of Eureka
2.	Electric Reliability Council	Western Systems Coordinating Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	220
5.	Design Electrical Rating (MWE Net)	65
6.	Date of Initial Criticality	February 16, 1963
7.	Date of Commercial Operation	August 1963
8.	Condenser Cooling Method	Circulating sea water
9.	Condenser Cooling Water Source	Pacific Ocean (Humboldt Bay)

Utility & Contractor Information

10.	Licensee	Pacific Gas & Electric Co.
11.	Corporate Address	77 Beale Street San Francisco, California 94106
12.	Corporate Contact	P. A. Crane, Jr. Asst. General Counsel
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	GE
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	V
17.	IE Branch Chief	J. L. Crews
18.	IE Section Chief	D. M. Sternberg
19.	IE Resident Inspector	J. D. Carlson
20.	Licensing Project Manager	V. L. Rooney
21.	Docket Number	50-133
22.	License Number/Date of Issue	DPR-7, 8/28/62
23.	Public Document Room	Humboldt County Library 636 F Street Eureka, California 95501

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

SYSTEMS AND COMPONENT PROBLEMS:

None.

FACILITY ITEMS (PLANS AND PROCEDURES):

The plant has been shut down since July 1976.

Several items remaining to be completed during the outage have been deferred pending resolution of the seismic issues at the plant site. A date for return to operations has not been established.

MANAGERIAL ITEMS:

None

PLANT STATUS:

Plant is shutdown.

LAST IE SITE INSPECTION DATE: 05/20-22/80

INSPECTION REPORT NO: 50-133/80-02

SUMMARY

INSPECTION SUMMARY

+ Inspection on May 20-22, 1980 (Report No. 50-133/80-02)
Areas Inspected: Routine, unannounced inspection of operations; organization and administration; training; calibration; followup on IE Bulletins and Circulars; and independent inspection. The inspection involved 34 inspector hours onsite by two NRC inspectors.

Results: No items of noncompliance or deviations were identified.

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT

NONE

LICENSE STATUS

Order issued 5/21/76 requiring Utility to prove seismic qualification before resuming operation of unit after outage of 7/2/76

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 8-4-80

REVIEWED BY NRR DL: V. Rooney **DATE:** 8/4/80

HUMBOLDT BAY

AVERAGE DAILY POWER LEVEL (MWe)

OPERATING STATUS

1. Docket Number: 50-133
2. Utility Data Prepared By: B. C. Getty (707) 443-2787
3. Licensed Thermal Power (MWt): 220
4. Nameplate Rating (Gross MWe): 77 X 0.85 = 65
5. Design Electrical Rating (Net MWe): 65
6. Maximum Dependable Capacity (Gross MWe): 0
7. Maximum Dependable Capacity (Net MWe): 0
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons

None

9. Power Level To Which Restricted, If Any (Net MWe): None
10. Reasons For Restrictions, If Any:

NO DECISION ON FUTURE OPERATION
SHUTDOWN INDEFINITELY

This ive

11. Hours In Reporting Period
12. Number Of Hours Reactor Was Cri
13. Reactor Reserve Shutdow
14. Hours Generato
15. Unit R
16. H)
17. erated (MWH)
18. Net gy Generated (MWH)
19. Unit S ce Factor
20. Unit Availability Factor
21. Unit Capacity Factor (Using MDC Net) 1/
22. Unit Capacity Factor (Using DER Net) 1/
23. Unit Forced Outage Rate
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

None

25. If Shut Down At End Of Report Period, Estimated Date of Startup: UNKNOWN

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
SEE SUMMARY BELOW									

SUMMARY

THE UNIT WAS SHUTDOWN ON 7/2/76 FOR SEISMIC MODIFICATIONS. THE UNIT REMAINS IN THE COLD SHUTDOWN CONDITION.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

INDIAN POINT 1

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	New York/Westchester 25 miles N of New York City
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	PWR/fossil superheat
4.	Power Level (MWT)	615
5.	Design Electrical Rating (MWE Net)	265
6.	Date of Initial Criticality	August 2, 1962
7.	Date of Commercial Operation	October 1962
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Hudson River

Utility & Contractor Information

10.	Licensee	Consolidated Edison Company
11.	Corporate Address	of New York, Inc. 4 Irving Place New York, New York
12.	Corporate Contact	P. Zarakas/Vice President
13.	Architect/Engineer	Utility
14.	Nuclear Steam Supply System	B&W
15.	Constructor	Utility

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Section Chief	H. B. KISTER
19.	IE Resident Inspector	T. Rebelowski (914-739-9360) FTS: 656-9220
20.	Licensing Project Manager	L. Olshan x27144
21.	Docket Number	50-03
22.	License Number/Date of Issue	DPR-5, 3/26/62
23.	Public Document Room	White Plains Public Library 100 Martine Avenue White Plains, New York 10601

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

```
    INSPECTION STATUS

    OTHER ITEMS OF INTEREST

    PLANT STATUS.

    Defueled.

  LAST IE SITE INSPECTION DATE:  7/24-27/79

        INSPECTION REPORT NO:  50-03/79-02

3.  SUMMARY

    INSPECTION SUMMARY

    None conducted.
```

LICENSE STATUS

The unit produced electricity to the expiration date (October 31, 1974) of delay granted by the AEC for installation of an ECCS system. The ECCS system was not subsequently installed and the unit was defueled on January 30, 1976.

On February 11, 1980, the NRC requested the utility to show cause as to why the operating authority of the license should not be revoked and why the licensee should not submit a plan within 120 days of this order to decommission the unit.

On June 19, 1980, an order was issued by the NRC to revoke the operating authority for Indian Point Station, Unit 1, as an electrical generating facility. Consolidated Edison Company of New York, Inc. is to submit a decommissioning plan for Unit 1 within 120 days.

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT

NONE

INDIAN POINT 1

AVERAGE DAILY POWER LEVEL (MWe)

OPERATING STATUS

1. Docket Number: 50-003
2. Utility Data Prepared By: L. J. Kawula (914)-694-6000 x209
3. Licensed Thermal Power (MWt): 615
4. Nameplate Rating (Gross MWe): 275
5. Design Electrical Rating (Net MWe): 265
6. Maximum Dependable Capacity (Gross MWe): 0
7. Maximum Dependable Capacity (Net MWe): 0
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons

None

9. Power Level To Which Restricted, If Any (Net MWe): None
10. Reasons For Restrictions, If Any:

This Month Yr. to

11. Hours In Reporting Period
12. Number Of Hours Reactor Was Critical
13. Reactor Reserve Shutdown Hours
14. Hours Generator On-Line
15. Unit Reserve Shutdown H
16. Gross Thermal E
17. Gross E
1

or

Factor (Using MDC Net) 1/

Capacity Factor (Using DER Net) 1/

23. Unit Forced Outage Rate
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

None

25. If Shut Down At End Of Report Period, Estimated Date of Startup: N/A

UNIT RETIRED AS GENERATING FACILITY. DECOMMISSIONING PLAN TO BE SUBMITTED BY LICENSEE

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
SEE SUMMARY BELOW									

SUMMARY

UNIT SHUTDOWN ON OCTOBER 31, 1974, ALL FUEL HAS BEEN UNLOADED; UNIT RETIRED AS A GENERATING FACILITY ON JUNE 19, 1980; DECOMMISSIONING PLAN TO BE SUBMITTED.

LEGEND

[1]Type:	[2]Reason:		[3]Method:	[4]Exhibit F:	[5]Exhibit H:
F--Forced S--Scheduled	A--Equipment Failure B--Maintenance or Test C--Refueling D--Regulatory Restriction	E--Operator Training & License Examination F--Administrative G--Operational Error H--Other	1--Manual 2--Manual Scram 3--Automatic Scram 4--Continuations 5--Load Reduction 9--Other	Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)	Same source.

INDIAN POINT 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	New York/Westchester 25 miles N of New York City
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2758
5.	Design Electrical Rating (MWE Net)	873
6.	Date of Initial Criticality	May 22, 1973
7.	Date of Commercial Operation	August 1973
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Hudson River

Utility & Contractor Information

10.	Licensee	Consolidated Edison Company of New York, Inc.
11.	Corporate Address	4 Irving Place New York, New York
12.	Corporate Contact	P. Zarakas, Vice President
13.	Architect/Engineer	UE&C
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	J. A. Jones/UE&C

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Senior Inspector	H. B. KISTER
19.	IE Resident Inspector	T. Rebelowski (914-739-9360) FTS: 656-9220
20.	Licensing Project Manager	L. Olshan (X27144)
21.	Docket Number	50-247
22.	License Number/Date of Issue	DPR-26, 10/19/71
23.	Public Document Room	White Plains Public Library 100 Martine Avenue White Plains, New York 10601

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

+ SYSTEMS AND COMPONENT PROBLEMS:

+ The licensee experienced a station blackout on June 3, due to grounding or loss of all incoming feeders. Licensee maintained natural circulation in core until restoration of power.

+ FACILITY ITEMS (PLANS AND PROCEDURES):

+ Unit 2 present plans call for Refueling Outage starting 1/1/81. Licensee has completed his actions to comply with the 120 day requirements of the Confirmatory Order issued February 11, 1980.
PLANT STATUS:

Unit at 100% power.

+ LAST IE SITE INSPECTION DATE: 6/22/80

INSPECTION REPORT NO: 50-247/80-08

SUMMARY

INSPECTION SUMMARY

+ 50-247/80-06 - May 13-16: Routine, onsite, regular and backshift inspection by a resident inspector (35 hours) of loss of coolant accident (LOCA) procedures, and training of licensed operators in the use of the LOCA procedures. No items of noncompliance were identified.

REPORTS RECEIVED FROM LICENSEE

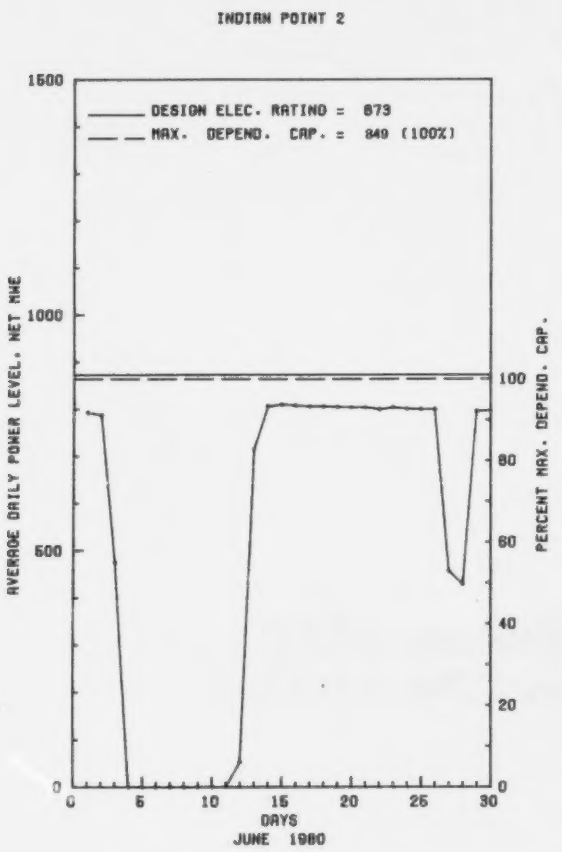
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-003/03L	04/17/80	05/16/80	Boron concentration in the boron injection tank found to be below TS requirements by one quarter of one percent.
80-004/03L	05/12/80	06/11/80	Seal injection line found leaking at socket weld connection.
80-006/99T	06/03/80	06/17/80	Loss of off-site power. Station blackout. Natural circulation of reactor coolant system.

REVIEWED BY OMPA LOEB: Philip A. Ross **DATE:** 08/04/80

REVIEWED BY NRR DL: W J Ross **DATE:** 8-5-80

INDIAN POINT 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-247
2. Utility Contact: E. EICH (914) 694-6000 @ I.P.
3. Licensed Thermal Power (MWt): 2758
4. Nameplate Rating (Gross MWe): 1126 x 0.9 = 1013
5. Design Electrical Rating (Net MWe): 873
6. Maximum Dependable Capacity (Gross MWe): 885
7. Maximum Dependable Capacity (Net MWe): 849
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): 824
10. Reasons for Restrictions, If Any:
REMOVAL OF #3 DISC. (GEN. END) ON #23 TURBINE ROTOR.

ITEM 21 CALCULATED BY USING A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	4,367.0	52,608.0*
12. Hours Reactor Was Critical	524.1	3,307.9	35,602.4
13. Reactor Reserve Shutdown Hours	.0	852.4	1,093.8
14. Hours Generator On-Line	492.2	3,218.3	34,607.5
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,310,304	8,435,479	89,268,005
17. Gross Electrical Energy (MWH)	395,010	2,555,460	27,688,546
18. Net Electrical Energy (MWH)	374,724	2,434,269	26,399,369
19. Unit Service Factor	68.4	73.7	65.8
20. Unit Availability Factor	68.4	73.7	65.8
21. Unit Capacity Factor (MDC Net)	61.3	65.7	58.3
22. Unit Capacity Factor (DER Net)	59.6	63.9	57.5
23. Unit Forced Outage Rate	31.6	11.8	8.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

*COMPUTED SINCE 7/1/74, THE DATE OF COMPLETION OF A 100 DAY – 100% POWER OPERATION TEST.

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
8	06/03/80	F	217.2	A	3	80-006	EA	ZZZZZZ	LOSS OF OFF-SITE POWER.
9	06/27/80	F	10.6	A	3		HA	GENERA	LOSS OF GENERATOR EXCITATION.

SUMMARY

THE UNIT OPERATED NEAR FULL POWER IN JUNE. ON 06/03/80 A LIGHTNING STRIKE BETWEEN SUBSTATIONS RESULTED IN ALL OFF-SITE POWER LOSS AND A REACTOR TRIP. DURING THE OUTAGE CONDENSOR TUBE LEAKS WERE REPAIRED AS WELL AS OTHER MAINTENANCE. ANOTHER TRIP OCCURRED ON 06/27/80 DUE TO A LOSS OF FIELD RELAY PROTECTION CAUSED BY A DEFECTIVE OVERCURRENT RELAY.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

INDIAN POINT 3

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	New York/Westchester 25 miles N of New York City
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	3025
5.	Design Electrical Rating (MWE Net)	965
6.	Date of Initial Criticality	April 6, 1976
7.	Date of Commercial Operation	August 30, 1976
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Hudson River

Utility & Contractor Information

10.	Licensee	Power Authority of the State of New York, Inc.
11.	Corporate Address	10 Columbus Circle New York, New York
12.	Corporate Contact	Peter N. Lyon - Senior Vice President
13.	Architect/Engineer	UE&C
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	WEDCO

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Section Chief	H. B. KISTER
19.	IE Resident Inspector	T. Rebelowski (914-739-9360) FTS 656-9220
20.	Licensing Project Manager	L. Olshan x27144
21.	Docket Number	50-286
22.	License Number/Date of Issue	DPR-64, 12/12/76
23.	Public Document Room	White Plains Public Library 100 Martine Avenue White Plains, New York 10601

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO TS 4.4.H, THE LICENSEE FAILED TO EVALUATE HIS FINDINGS OF SURVEILLANCE DOCUMENT 3 PT-A 2 CONTAINMENT STRUCTURAL INSPECTION. (7929 INFRACTION)

CONTRARY TO TS SECTION 6.8.1.C & ADMIN PROCEDURE A-19, SURVEILLANCE TESTS WERE REVIEWED & ACCEPTED AS SATISFACTORY BY THE LICENSEE WITH DATA MISSING, PROCEDURE CHANGES NOT INCORPORATED & DISCREPANT CONDITIONS NOT PROPERLY EVALUATED & RESOLVED. (7929 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

MANAGERIAL ITEMS:

- Mr. S. Zulla has been appointed PASNY Resident Manager at Indian Point 3.
- FACILITY ITEMS (PLANS AND PROCEDURES):
- Licensee has completed his actions to comply with the 120 day requirements of the Confirmatory Order issued February 11, 1980.

PLANT STATUS:

Unit at 90% power.

- LAST IE SITE INSPECTION DATE: 6/26/80
- INSPECTION REPORT NO: 50-286/80-09

SUMMARY

INSPECTION SUMMARY

- 50-286/80-01 - Jan 20 - Mar 15: Routine unannounced inspection by the Resident Inspector and two regional based inspectors (108 hours) of plant operations including: log and record review; tour of the facility, observation of physical security, observation and review of plant trips and safety challenges, preparation for plant startup following refueling, startup testing-refueling, lessons learned verification, licensee's action in response to Immediate Action Letter 80-03, review of minor seismic event and personnel background checks. No items of noncompliance were identified.

REPORTS RECEIVED FROM LICENSEE

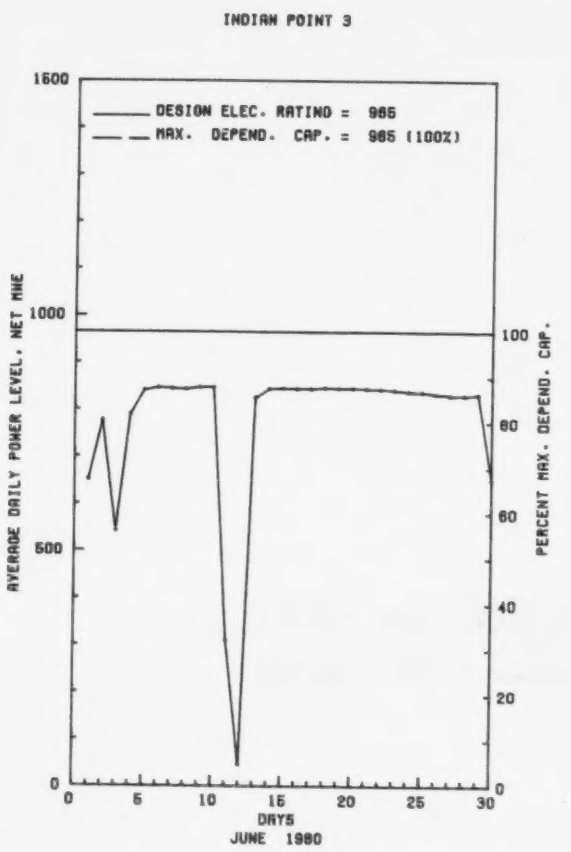
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
NONE				

REVIEWED BY OMPA LOEB: DATE: 8/4/80

REVIEWED BY NRR DL: W J Ross DATE: 8/5/80

INDIAN POINT 3

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-286
2. Utility Contact: C. Connell (914) 739-8200 x242
3. Licensed Thermal Power (MWt): 3025
4. Nameplate Rating (Gross MWe): 1126 x 0.9 = 1013
5. Design Electrical Rating (Net MWe): 965
6. Maximum Dependable Capacity (Gross MWe): 1000
7. Maximum Dependable Capacity (Net MWe): 965
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): 917
10. Reasons for Restrictions, If Any: REMOVED TURBINE BLADING

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	4,367.0	33,624.0
12. Hours Reactor Was Critical	702.2	3,048.5	24,633.4
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	681.6	2,614.3	23,647.1
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,996,621	5,807,257	61,911,724
17. Gross Electrical Energy (MWH)	577,820	1,622,500	19,987,601
18. Net Electrical Energy (MWH)	554,358	1,536,220	19,179,657
19. Unit Service Factor	94.7	59.9	70.3
20. Unit Availability Factor	94.7	59.9	70.3
21. Unit Capacity Factor (MDC Net)	79.8	36.5	59.1
22. Unit Capacity Factor (DER Net)	79.8	36.5	59.1
23. Unit Forced Outage Rate	5.3	20.3	6.3

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): TURBINE OUTAGE OCTOBER - 80.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
21	06/03/80	F	.0	A	4		HH	VALVEX	#32 HEATER DRAIN PUMP TRIPPED. REDUCED LOAD TO SECURE DUMPING TO CONDENSER FOR CONSIDERATION OF STEAM IMPINGEMENT ON TUBES. REPAIRED ASSOCIATED REGULATOR VALVE TO HEATER DRAIN PUMP.
22	06/11/80	F	33.4	A	3		EA	RELAYX	MISOPERATION OF NGA RELAY AT SUBSTATION IN BUCHANAN CAUSING DIRECT TRIP OF PLANT. MODIFIED RELAY.
23	06/30/80	F	5.0	A	3		HH	VALVEX	TRIPPED ON LOW STEAM GENERATOR LEVEL DUE TO LOSS OF MAIN BOILER FEEDWATER PUMPS AS A RESULT OF PERTURBATIONS IN MAIN BOILER FEEDWATER CONTROL OIL SYSTEM WHEN SHIFTING OIL PUMPS. PUMP CHECK VALVE APPEARED TO HANG UP. CYCLED ASSOCIATED OIL PUMP CHECK VALVE SEVERAL TIMES AND VALVE OPERATED PROPERLY.

SUMMARY

THE UNIT OPERATED NEAR FULL POWER EXCEPT FOR TRIPS ON 06/11/80, DUE TO SUBSTATION TRIP AND ON 06/30/80 DUE TO LOW STEAM GENERATOR LEVEL. A REDUCTION OCCURRED ON 06/03/80 DUE TO A HEATER DRAIN PUMP TRIP.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

KEWAUNEE

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Wisconsin/Kewaunee 27 miles E of Green Bay
2.	Electric Reliability Council	Mid-America Interpool Network
3.	Type of Reactor	PWR
4.	Power Level (MWT)	1650
5.	Design Electrical Rating (MWE Net)	535
6.	Date of Initial Criticality	March 7, 1974
7.	Date of Commercial Operation	June 1974
8.	Condenser Cooling Method	Once through condenser and return to Lake Michigan
9.	Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10.	Licensee	Wisconsin Public Service Corp.
11.	Corporate Address	P. O. Box 1200 Green Bay, Wisconsin 54305
12.	Corporate Contact	E. R. Mathews Power Supply & Eng.
13.	Architect/Engineer	Fluor Power Services Corp.
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Pioneer Services & Eng. Co.

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	R. F. Warnick
19.	IE Resident Inspector	R. L. Nelson
20.	Licensing Project Manager	R. B. Licciardo X28362
21.	Docket Number	50-305
22.	License Number/Date of Issue	DPR-43 issued 12/21/73
23.	Public Document Room	Kewaunee Public Library 822 Juneau Street Kewaunee, Wisconsin 54216

INSPECTION STATUS

ENFORCEMENT STATUS

BY NRC LETTER DATED NOVEMBER 5, 1979, TO MR. CORDELL REED, CHAIRMAN, WESTINGHOUSE OWNER'S GROUP, MR. D. F. ROSS, JR., DIRECTOR BULLETINS AND ORDERS TASK FORCE, PROMULGATED THE STAFF'S EVALUATION OF THE WESTINGHOUSE SMALL BREAK LOSS OF COOLANT ACCIDENT OPERATOR GUIDELINES, CLARIFIED VARIOUS COMMITMENTS ON THE PART OF THE OWNERS GROUP, AND ESTABLISHED DECEMBER 31, 1979, AS THE DATE FOR IMPLEMENTING THESE PROCEDURAL GUIDELINES IN THE OWNER'S PLANTS. CONTRARY TO THE ABOVE, INSPECTION INDICATES THAT PROCEDURES DO NOT REFLECT THE INFORMATION IN THE GUIDELINES. PROCEDURES ARE DEFICIENT IN THE FOLLOWING AREAS: (A) INITIAL SYMPTOMS AND IMMEDIATE OPERATOR ACTIONS HAVE NOT BEEN INCORPORATED IN AN EMERGENCY PROCEDURE, RATHER IT'S IN AN ABNORMAL PROCEDURE; (B) REQUIREMENT TO VERIFY NATURAL CIRCULATION IS NOT IN AN EMERGENCY PROCEDURE; (C) MOST OF THE CAUTIONS AND NOTES IN THE GUIDELINES ARE NOT INCORPORATED IN EMERGENCY PROCEDURES; (D) GUIDELINE REQUIREMENT TO INCORPORATE STEPS TO BE TAKEN IF A LOSS OF OFFSITE POWER OCCURS SUBSEQUENT TO A SAFETY INJECTION HAS NOT BEEN INCORPORATED IN SMALL-BREAK LOSS-OF-COOLANT ACCIDENT PROCEDURES. (8006 DEVIATION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

FACILITY ITEMS

The plant was shutdown May 9, 1980 for a scheduled 6 week refueling outage. (80-10)

PLANT STATUS

+ The plant returned to power operation on June 24, 1980, following a 6 week refueling outage.

+LAST IE SITE INSPECTION DATE: May 7 and 19-23, 1980

+ INSPECTION REPORT NO: 80-10

SUMMARY

INSPECTION SUMMARY

+ Inspection on May 13-15 and 28-30, (80-09): Review of inservice inspection program, procedures, observation of nondestructive examinations, and review of documentation. The inspection involved 64 inspector hours onsite by two NRC inspectors. Of the four areas examined, one apparent item of noncompliance was identified in one area (failure to train and qualify personnel in accordance with ANSI N45.2.6).

+ Inspection on May 7 and 19-23, (80-10): Routine, announced inspection of licensee event report followup, IE Bulletin followup, IE Circular followup, receipt of new fuel, preparation for refueling, refueling activities, maintenance refueling, and independent inspection. The inspection involved 33 inspector hours onsite by one NRC inspector. The inspection was accompanied by two NRC inspectors in-training. The management meeting involved 3 hours onsite by two NRC personnel. They were accompanied by two NRC inspectors in-training. Of the eight areas inspected, no items of noncompliance were identified in seven areas. Two items of noncompliance were identified in one area. (failure to follow procedures; and failure to report within one hour).

REPORTS RECEIVED FROM LICENSEE

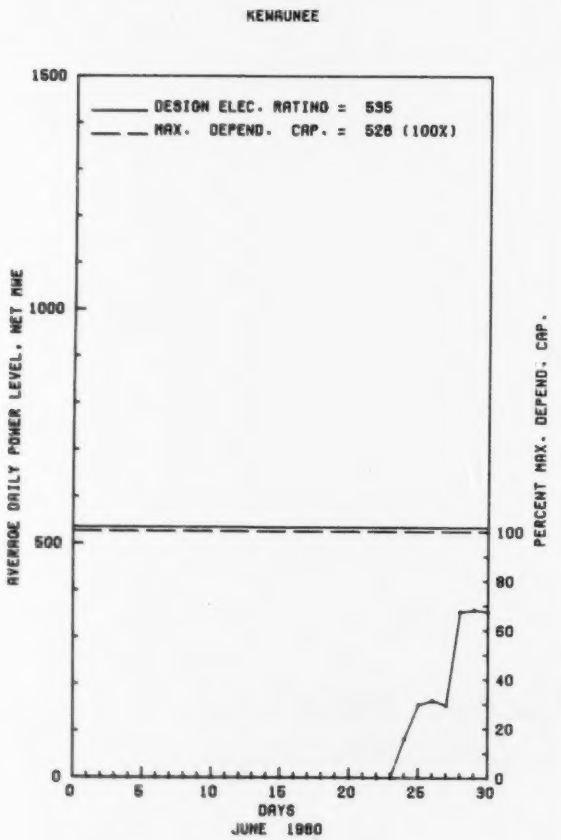
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-21/ 03L-0	03/28/80	05/23/80	Monthly vibration test was not performed within required time interval.
80-22/ 03L-0	05/19/80	06/16/80	One of three pressurizer pressure transmitters out of calibration.
80-23/ 04L-0	05/27/80	06/16/80	One channel of the SFIS out of calibration.

REVIEWED BY OMPA LOEB: Earl Hill **DATE:** 8/4/80

REVIEWED BY NRR DL: **DATE:** 8/6/80

KEWAUNEE

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-305
2. Utility Contact: G.RUITER (414) 388-2560 X207
3. Licensed Thermal Power (MWt): 1650
4. Nameplate Rating (Gross MWe): 622 x 0.9 = 560
5. Design Electrical Rating (Net MWe): 535
6. Maximum Dependable Capacity (Gross MWe): 545
7. Maximum Dependable Capacity (Net MWe): 526
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

ITEM 21 CALCULATED BY USING A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	4,367.0	52,968.0
12. Hours Reactor Was Critical	220.5	3,072.2	44,074.7
13. Reactor Reserve Shutdown Hours	.0	152.7	2,330.5
14. Hours Generator On-Line	152.5	2,987.0	42,983.0
15. Unit Reserve Shutdown Hours	.0	.0	10.0
16. Gross Thermal Energy (MWH)	137,929	4,724,040	65,875,163
17. Gross Electrical Energy (MWH)	42,400	1,566,700	21,750,500
18. Net Electrical Energy (MWH)	39,041	1,494,198	20,693,276
19. Unit Service Factor	21.2	68.4	81.1
20. Unit Availability Factor	21.2	68.4	81.2
21. Unit Capacity Factor (MDC Net)	10.3	65.0	74.5
22. Unit Capacity Factor (DER Net)	10.1	64.0	73.0
23. Unit Forced Outage Rate	.0	8.7	5.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
4	05/09/80	S	559.0	C	4		ZZ	ZZZZZZ	COMPLETED CYCLE V-VI REFUELING OUTAGE ON JUNE 24, 1980.
5	06/26/80	S	8.5	B	1		HA	TURBIN	A SHORT OUTAGE WAS TAKEN TO ADJUST BALANCE WEIGHTS ON THE TURBINE.

SUMMARY

FOLLOWING REFUELING OUTAGE, THE UNIT WAS RETURNED TO SERVICE ON 06/24/80. THE UNIT OPERATED NORMALLY WITH AN OUTAGE ON 06/26/80 TO ADJUST BALANCE ON TURBINES.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram.
3--Automatic Scram.
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

LACROSSE

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Wisconsin/Vernon 19 miles S of LaCrosse
2.	Electric Reliability Council	Mid-Continent Area Reliability Coordination Agreement
3.	Type of Reactor	BWR
4.	Power Level (MWT)	165
5.	Design Electrical Rating (MWE Net)	50
6.	Date of Initial Criticality	July 11, 1967
7.	Date of Commercial Operation	November 1, 1969
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Mississippi River

Utility & Contractor Information

10.	Licensee	Dairyland Power Cooperative
11.	Corporate Address	2615 East Avenue South La Crosse, Wisconsin
12.	Corporate Contact	F. Linder
13.	Architect/Engineer	Sargent & Lundy
14.	Nuclear Steam Supply System	Allis-Chalmers
15.	Constructor	Maxson Construction

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. F. HEISHMAN
18.	IE Section Chief	D. C. BOYD
19.	IE Resident Inspector	W. Forney
20.	Licensing Project Manager	J. J. Shea
21.	Docket Number	50-409
22.	License Number/Date of Issue	DPR-45 issued 8/28/73
23.	Public Document Room	LaCrosse Public Library 800 Main Street LaCrosse, Wisconsin 54601

INSPECTION STATUS

ENFORCEMENT STATUS

10 CFR SECTION 50.72 REQUIRES, IN PART, LICENSEES OF NUCLEAR POWER REACTORS LICENSED UNDER SECTION 50.21 OR SECTION 50.22 TO NOTIFY THE NRC OPERATIONS CENTER AS SOON AS POSSIBLE AND IN ALL CASES WITHIN ONE HOUR BY TELEPHONE OF ANY EVENT RESULTING IN AUTOMATIC ACTUATION OF THE REACTOR PROTECTION SYSTEM. CONTRARY TO THE ABOVE REQUIREMENT, AT 7:21 P.M. ON APRIL 28, 1980, THE PLANT EXPERIENCED A REACTOR TRIP, DURING STARTUP AT LOW POWER, DUE TO A LOW REACTOR VESSEL LEVEL. THIS EVENT WAS NOT REPORTED WITHIN THE ONE HOUR TIME LIMIT AND NRC WAS NOT INFORMED OF THE OCCURRENCE UNTIL APPROXIMATELY 6:15 A.M. ON APRIL 29, 1980, DURING A ROUTINE STATUS CALL FROM THE NRC OPERATIONS CENTER. (8001 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

FACILITY ITEMS

+ During the month of April the Plant was shutdown for approximately 3 weeks to accomplish short-term TMI modifications as required by NUREG 0578. (80-01)

PLANT STATUS

Plant is operating routinely.

+LAST IE SITE INSPECTION DATE: March 19-20 and May 5-9, 1980

+ INSPECTION REPORT NO: 80-01

INSPECTION SUMMARY

+ Inspection on December 27-28, (79-24): Nonroutine, announced inspection of licensee actions taken in response to IE Bulletin 79-19, "Packaging of Low-Level Radioactive Waste for Transport and Burial." The inspection involved 12 inspector hours onsite by one NRC inspector. No items of noncompliance or deviations were identified.

+ Inspection on March 19-20 and May 5-9, (80-01): Routine, unannounced inspection of the licensee's quality assurance program; design, design changes, and modifications; plant operations; TMI short-term modifications; and followup actions relative to previous items of noncompliance, IE Bulletins, Licensee Event Reports, and open inspection items. The inspection involved 105 inspector hours onsite by three NRC inspectors. Of the eight areas inspected, no items of noncompliance or deviations were found in seven areas; one item of noncompliance was found in one area (failure to make prompt telephone notification to NRC Operations Center of significant event).

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
NONE				

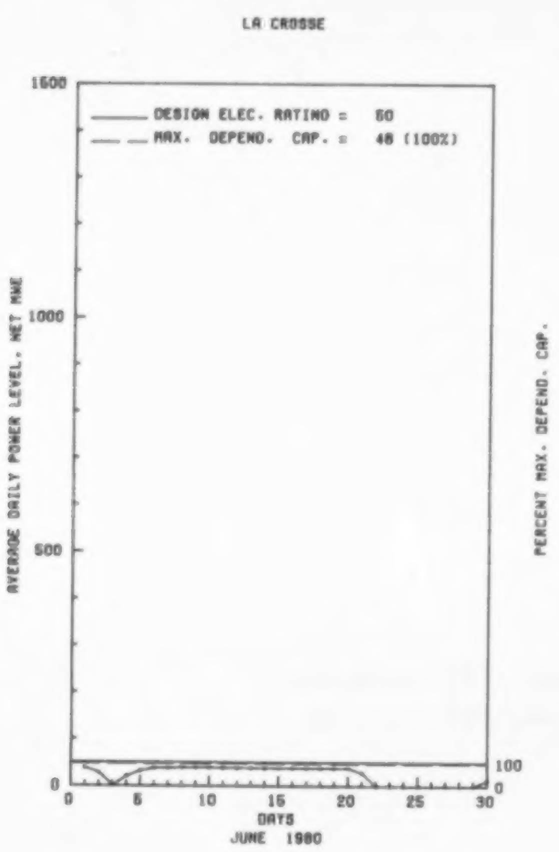
REVIEWED BY OMPA LOEB: John J. Rizzo DATE: 8-4-80

REVIEWED BY NRR DL: J Shea DATE: 8-6-80

GPO 873-667

LACROSSE

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-409
2. Utility Contact: L. S. Goodman (608) 689-2331
3. Licensed Thermal Power (MWt): 165
4. Nameplate Rating (Gross MWe): 76.8 x 0.85 =65
5. Design Electrical Rating (Net MWe): 50
6. Maximum Dependable Capacity (Gross MWe): 50
7. Maximum Dependable Capacity (Net MWe): 48
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	4,367.0	93,482.0
12. Hours Reactor Was Critical	539.5	3,621.9	61,428.0
13. Reactor Reserve Shutdown Hours	.0	.0	478.0
14. Hours Generator On-Line	502.1	3,493.0	56,498.0
15. Unit Reserve Shutdown Hours	.0	.0	79.0
16. Gross Thermal Energy (MWH)	65,131	469,526	7,754,305
17. Gross Electrical Energy (MWH)	18,839	137,569	2,336,701
18. Net Electrical Energy (MWH)	17,357	128,587	2,161,322
19. Unit Service Factor	69.7	80.0	60.4
20. Unit Availability Factor	69.7	80.0	60.5
21. Unit Capacity Factor (MDC Net)	50.2	61.3	48.2
22. Unit Capacity Factor (DER Net)	48.2	58.9	46.2
23. Unit Forced Outage Rate	27.8	7.1	6.4

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): ESTIMATED REFUELING OUTAGE, OCTOBER 10, 1980 (6 WEEKS).
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-04	06/02/80	S	24.2	E	1		ZZ	ZZZZZZ	LICENSE EXAMINATIONS.
80-05	06/21/80	F	193.7	A	1		RB	CRDRVE	LACBWR WAS SHUTDOWN TO REPAIR A SEAL LEAK IN UPPER CONTROL ROD DRIVE MECHANISM NO. 24. THE SEAL ON FORCED CIRCULATION PUMP 1A WAS ALSO REPAIRED WHILE THE PLANT WAS SHUTDOWN.

SUMMARY

THE UNIT RAN NORMALLY EXCEPT FOR OUTAGES ON 06/02/80 DUE TO LICENSE EXAMINATIONS AND ON 06/21/80 TO REPAIR A SEAL LEAK TO THE UPPER CONTROL ROD DRIVE MECHANISM.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenence or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

MAINE YANKEE

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1. Location (state/county) (nearest population center)	Maine/Lincoln 10 miles N of Bath (near Wiscasset)
2. Electric Reliability Council	Northeast Power Coordinating Council
3. Type of Reactor	PWR
4. Power Level (MWT)	2630
5. Design Electrical Rating (MWE Net)	825
6. Date of Initial Criticality	October 23, 1972
7. Date of Commercial Operation	December 28, 1972
8. Condenser Cooling Method	Once Through
9. Condenser Cooling Water Source	Back River (Atlantic)

Utility & Contractor Information

10. Licensee	Maine Yankee Atomic Power Co.
11. Corporate Address	20 Turnpike Road Westboro, Massachusetts 01581
12. Corporate Contact	D. W. Edwards, Assistant to Vice President
13. Architect/Engineer	Stone & Webster
14. Nuclear Steam Supply System	Combustion Engineering
15. Constructor	Stone & Webster

Regulatory Information

16. IE Region responsible	I
17. IE Branch Chief	E. Brunner
18. IE Section Chief	T. T. Martin
19. IE Resident Inspector	W. J. LAZARUS (X5258)
20. Licensing Project Manager	G. Requa (X28478)
21. Docket Number	50-309
22. License Number/Date of Issue	DPR-36, 9/15/72
23. Public Document Room	Wiscasset Public Library High Street Wiscasset, Maine 04578

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

+ The No. one steam generator non-return valve was repaired. The plant was returned to full power operation. Chemical cleaning of the reactor coolant system to reduce core pressure drop was apparently successful and the plant began the use of lithium in the reactor coolant system to reduce corrosion and crud buildup.

PLANT STATUS:

+ The plant returned to power operation on June 13 following a maintenance outage.

+ LAST IE SITE INSPECTION DATE: 6/30/80

+ INSPECTION REPORT NO: 50-309/80-09

SUMMARY

INSPECTION SUMMARY

+ 50-309/80-06 - Apr 7-11 and 21-24: Routine unannounced inspection by two regional based inspectors (77 hours) of plant operations, including a tour of accessible areas and a review of logs and records; review of licensee action on selected previous inspection findings and selected IE Bulletins and Circulars; on site followup of selected Licensee Event Reports; and observation of preparation and shipment of radioactive waste. No items of noncompliance were identified.

+ 50-309/80-07 - May 5-9: Routine unannounced inspection by one regional based inspector (18 hours) of plant operations; followup on selected Licensee Event Reports; and licensee action to limit on-line containment purging and ventilation. No items of noncompliance were identified.

+ 50-309/80-08 - May 19-22: Unannounced Physical Protection inspection included a review of Security Plan; Security Program Audit; Security Organization; Protected Area Physical Barriers; Vital Area Physical Barriers; Lighting; Access Control (Identification, Authorization and Badging); Access Control (Search); Access Control (Escorting); Access Control (Vital Areas); Alarm Stations; Detection Aids; Assessment Aids, Communications; Testing and Maintenance; Compensatory Measures; Power Supply; Response; Locks, Keys and Combinations; Records and Reports; and Followup on IE Bulletin No. 79-16. Three items of noncompliance were identified: Explosives detector inoperable; Unlocked door; Vital Area lock defective.

REPORTS RECEIVED FROM LICENSEE

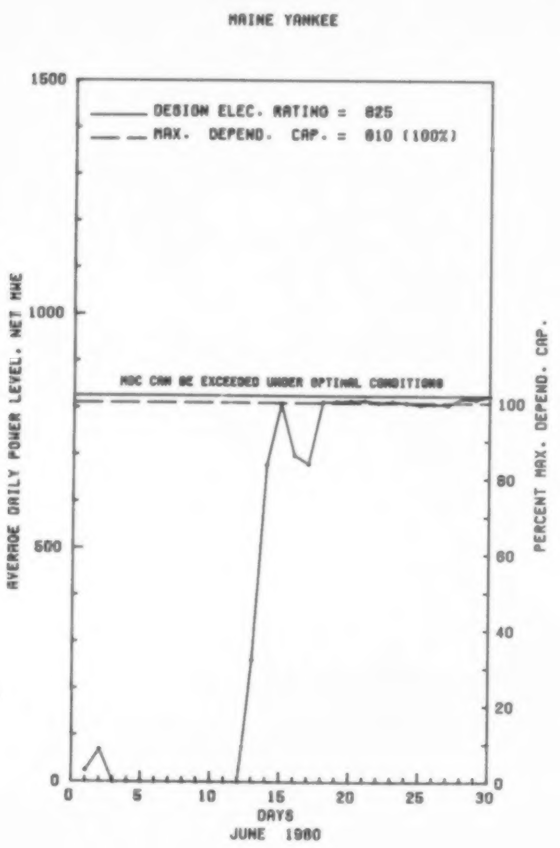
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-009/03L	04/08/80	05/02/80	LPSI header stop valve malfunction
80-011/03L	04/22/80	05/20/80	RPS logic matrix relay malfunction

REVIEWED BY OMPA LOEB: ML Beebe **DATE:** 08/04/80

REVIEWED BY NRR DL: G. Requa **DATE:** 08/05/80

MAINE YANKEE

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-309
2. Utility Contact: F.C. BEERS (617) 366-9011
3. Licensed Thermal Power (MWt): 2630
4. Nameplate Rating (Gross MWe): 900 x 0.9 = 810
5. Design Electrical Rating (Net MWe): 825
6. Maximum Dependable Capacity (Gross MWe): 850
7. Maximum Dependable Capacity (Net MWe): 810
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:

NONE

9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:

NONE

CAPACITY FACTORS CALCULATED WITH A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	4,367.0	66,995.6
12. Hours Reactor Was Critical	484.5	2,605.0	53,814.9
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	448.5	2,429.6	51,947.3
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,046,555	5,233,568	110,475,651
17. Gross Electrical Energy (MWH)	346,260	1,710,510	36,303,690
18. Net Electrical Energy (MWH)	330,935	1,646,135	34,473,321
19. Unit Service Factor	62.3	55.6	77.5
20. Unit Availability Factor	62.3	55.6	77.5
21. Unit Capacity Factor (MDC Net)	56.7	46.5	66.7
22. Unit Capacity Factor (DER Net)	55.7	45.7	64.5
23. Unit Forced Outage Rate	36.3	12.5	7.2

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

NONE

25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
8-80-5	05/30/80	S	16.1	H	4		RC	ZZZZZZ	SHUTDOWN TO ACCOMPLISH CHEMICAL CLEANING OF THE RCS TO REMOVE CORE CRUD.
9-80-5	06/02/80	F	255.4	A	1	80-016/03L	HB	VALVEX	FAILURE OF #1 S/G NRV DISC. VALVE WAS DISASSEMBLED, DISC REMOVED AND REPAIRED.
	06/16/80	F	42.0	A	5		HF	HTEXCH	CONDENSER TUBE LEAK IN THE "C" WATERBOX 1 TUBE PLUGGED AND 1 TUBE RE-ROLLED.

SUMMARY

THE UNIT BEGAN THE MONTH SHUTDOWN FOR CHEMICAL CLEANING. IT WAS BROUGHT BACK TO SERVICE ON 06/01/80. ON 06/02/80 THROUGH 06/13/80 IT WAS SHUTDOWN DUE TO NRV DISC FAILURE. THE UNIT RAN NORMALLY FOR THE REST OF THE MONTH WITH A REDUCTION ON 06/16/80 DUE TO CONDENSER TUBE LEAK.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram.
3--Automatic Scram.
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

MILLSTONE 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1. Location (state/county) (nearest population center)	Connecticut/New London 5 miles SW of New London
2. Electric Reliability Council	Northeast Power Coordinating Council
3. Type of Reactor	BWR
4. Power Level (MWT)	2011
5. Design Electrical Rating (MWE Net)	660
6. Date of Initial Criticality	October 26, 1970
7. Date of Commercial Operation	March 1971
8. Condenser Cooling Method	Once Through
9. Condenser Cooling Water Source	Long Island Sound

Utility & Contractor Information

10. Licensee	Northeast Nuclear Energy Company
11. Corporate Address	P. O. Box 270 Hartford, Connecticut
12. Corporate Contact	W. G. Counsil, Vice President
13. Architect/Engineer	Ebasco
14. Nuclear Steam Supply System	GE
15. Constructor	Ebasco

Regulatory Information

16. IE Region responsible	I
17. IE Branch Chief	E. Brunner
18. IE Section Chief	R. KEIMIG
19. IE Resident Inspector	J. SHEDLOSKY (203-447-3179) FTS: 642-7287
20. Licensing Project Manager	J. J. Shea
21. Docket Number	50-245
22. License Number/Date of Issue	DPR-21, 10/7/70
23. Public Document Room	Waterford Public Library Rope Ferry Road Route 156 Waterford, Connecticut 06385

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEM AND COMPONENT PROBLEMS:

- The plant was returned to power operation on 6/24 following an outage on 5/31 to repair main turbine extraction steam lines. During the outage repairs were made to the LPCI 'A' Containment Penetration Pipe restraint which was discovered in a degraded condition on 6/13.
- FACILITY ITEMS (PLANS AND PROCEDURES):
- An eight to ten week refueling outage is scheduled to commence on 9/19.

PLANT STATUS:

- The plant returned to power operation on 6/24. A trip occurred on 6/25 due to a pressure spike caused by the turbine control system.
- LAST IE SITE INSPECTION DATE: 6/27/80
- INSPECTION REPORT NO: 50-245/80-10

SUMMARY

INSPECTION SUMMARY

- 50-245/80-06 - Mar 30 - May 3: Routine, onsite, regular and backshift inspection by two resident inspectors and one regional based inspector (78 hours) of the control room and the accessible portions of the reactor, turbine, radioactive waste, gas turbine generator, and intake buildings; radiation protection; physical security; fire protection; plant operating records; surveillance of pipe supports; surveillance testing; calibration; maintenance; core power distribution limits; and reporting to the NRC. No items of noncompliance were identified.
- 50-245/80-07 - May 1-31: Routine, onsite, regular and backshift inspection by two resident inspectors (47 hours). Areas inspected included the control room and the accessible portions of the reactor, turbine, radioactive waste, gas turbine generator, and intake buildings; radiation protection; physical security; fire protection; plant operating records; surveillance testing; calibration; maintenance; core power distribution limits; and reporting to the NRC. No items of noncompliance were identified.

REPORTS RECEIVED FROM LICENSEE

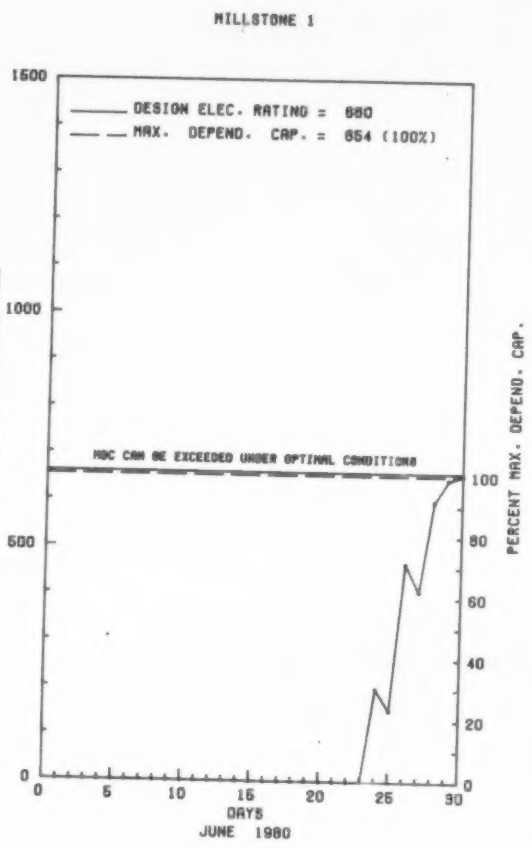
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-007/03L	05/16/80	06/13/80	Failure to complete semi-annual surveillance testing of cable spread area smoke detection system

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 8-4-80

REVIEWED BY NRR DL: J J Shea **DATE:** 8-6-80

MILLSTONE 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-245
2. Utility Contact: R. Young (203) 447-1792 x475
3. Licensed Thermal Power (MWt): 2011
4. Nameplate Rating (Gross MWe): 735 x 0.9 = 662
5. Design Electrical Rating (Net MWe): 660
6. Maximum Dependable Capacity (Gross MWe): 684
7. Maximum Dependable Capacity (Net MWe): 654
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	4,367.0	84,047.0
12. Hours Reactor Was Critical	165.5	3,793.0	64,178.0
13. Reactor Reserve Shutdown Hours	557.8	577.3	1,512.3
14. Hours Generator On-Line	149.0	3,772.5	61,868.4
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	227,825	6,177,714	110,225,659
17. Gross Electrical Energy (MWH)	78,847	2,097,247	37,328,643
18. Net Electrical Energy (MWH)	72,230	1,998,695	35,638,477
19. Unit Service Factor	20.7	86.4	73.6
20. Unit Availability Factor	20.7	86.4	75.6
21. Unit Capacity Factor (MDC Net)	15.3	70.0	64.8
22. Unit Capacity Factor (DER Net)	15.2	69.3	64.2
23. Unit Forced Outage Rate	8.1	.3	15.8

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): ANNUAL REFUEL. & MAIN OUTAGE SCHED. 9/27/80 - 8-10 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
04	05/30/80	S	360.0	A	4		HJ	TURBIN	UNIT WAS REMOVED FROM SERVICE TO INSPECT AND REPAIR AS NECESSARY AN EXTRACTION STEAM PIPING LEAK OFF L.P. TURBINE.
04A	06/16/80	S	197.8	H	4	80-10/3L	SF	SUPORT	QUESTIONABLE INTEGRITY OF LPCI SUBSYSTEM 'B' INJECTION PIPING AT PENETRATION X-45 MADE REPAIRS NECESSARY. THIS EXTENDED NO. 06 SHUTDOWN FOR THE HOURS INDICATED (DURATION).
05	06/25/80	F	13.2	A	3		CC	XXXXXX	ELECTRIC PRESSURE REGULATOR MALFUNCTION INDUCED APRM SCRAM. THE PRESSURE CONTROL WAS TRANSFERRED TO MECHANICAL.

SUMMARY

THE UNIT BEGAN THE PERIOD WITH A MAINTENANCE OUTAGE TO REPAIR STEAM PIPING LEAKS AND LPCI SUBSYSTEM. IT WAS BROUGHT BACK ON LINE ON 06/24/80 AND RAN NORMALLY EXCEPT FOR A SCRAM ON 06/25/80 DUE TO ELECTRIC PRESSURE REGULATOR MALFUNCTION.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

MILLSTONE 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Connecticut/New London 5 miles SW of New London
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2700
5.	Design Electrical Rating (MWE Net)	870
6.	Date of Initial Criticality	October 17, 1975
7.	Date of Commercial Operation	December 26, 1975
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Long Island Sound

Utility & Contractor Information

10.	Licensee	Northeast Nuclear Energy Company
11.	Corporate Address	P. O. Box 270 Hartford, Connecticut
12.	Corporate Contact	W. G. Counsil, Vice President
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Combustion Engineering
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	IE Section Chief	E. McCabe
19.	IE Resident Inspector	J. T. SHEDLOSKY (203-447-3179) FTS: 642-7287
20.	Licensing Project Manager	R. Zimmerman E. L. Conner, Jr.
21.	Docket Number	50-336
22.	License Number/Date of Issue	DPR-65, 8/1/75
23.	Public Document Room	Waterford Public Library Rope Ferry Road Route 156 Waterford, Connecticut 06385

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

+ The plant returned to power operation on 6/24 following an outage on 5/8 to upgrade pipe restraints found to have less than acceptable factors of safety.

+ FACILITY ITEMS (PLANS AND PROCEDURES):

+ A nine week refueling outage is scheduled to commence on 8/2/80.

PLANT STATUS:

+ The plant was returned to power on 6/24.

+ LAST IE SITE INSPECTION DATE: 6/27/80

+ INSPECTION REPORT NO: 50-336/80-08

SUMMARY

INSPECTION SUMMARY

+ 50-336/80-04 - Mar 30 - May 3: Routine, onsite, regular and backshift inspection by two resident inspectors and one regional based inspector (131 hours) of the control room; the enclosure, auxiliary, turbine and intake buildings; and the condensate polishing facility; radiation protection; physical security; fire protection; plant operating records; surveillance of pipe supports; surveillance testing; calibration; maintenance; core power distribution limits; and reporting to the NRC. Three items of noncompliance were identified: (Failure to wear dosimetry in accordance with H.P. procedures; failure to meet snubber surveillance requirements; failure to comply with security procedures).

+ 50-336/80-05 - May 1-31: Routine, onsite, regular and backshift inspection by two resident inspectors (65 hours). Areas inspected included the control room, primary containment enclosure, auxiliary, turbine and intake buildings; and the condensate polishing facility; radiation protection; physical security; fire protection; plant operating records; surveillance testing; calibration; maintenance; core power distribution limits; and reporting to the NRC. No items of noncompliance were identified.

REPORTS RECEIVED FROM LICENSEE

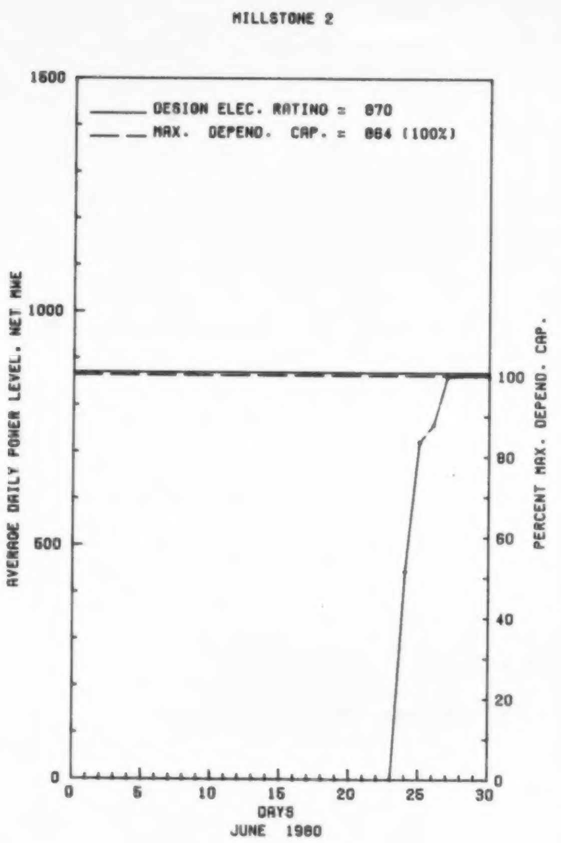
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-018/01T	05/08/80	05/22/80	Several service water piping support hangers were identified as not having a factor of safety greater than two. The plant was placed in cold shutdown to effect repairs.
80-020/03L	04/29/80	05/29/80	Stuck open pressurizer spray valve resulted in reactor coolant pressure less than that specified for DNB margin. Valve failure due to a loose valve seat.
80-021/03L	05/06/80	05/06/80	Diesel generator breaker reclosed following manual tripping. Generator returned to bus out of phase. The failure was due to a loose bolt on latch in spring charging system.

REVIEWED BY OMPA LOEB: MR Beebe **DATE:** 08/04/80

REVIEWED BY NRR DL: Cheng-Yang Li **DATE:** 8/6/80

MILLSTONE 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-336
2. Utility Contact: G. H. Howlett (203) 447-1791 x364
3. Licensed Thermal Power (MWt): 2700
4. Nameplate Rating (Gross MWe): 1011 x 0.9 = 910
5. Design Electrical Rating (Net MWe): 870
6. Maximum Dependable Capacity (Gross MWe): 895
7. Maximum Dependable Capacity (Net MWe): 864

CAPACITY FACTORS CALCULATED WITH A WEIGHTED AVERAGE

8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	4,367.0	39,575.0
12. Hours Reactor Was Critical	173.0	3,182.6	28,261.3
13. Reactor Reserve Shutdown Hours	.0	.0	2,072.4
14. Hours Generator On-Line	163.0	3,096.7	26,916.9
15. Unit Reserve Shutdown Hours	.0	.0	335.4
16. Gross Thermal Energy (MWH)	404,590	8,161,936	65,860,564
17. Gross Electrical Energy (MWH)	134,199	2,671,577	21,256,222
18. Net Electrical Energy (MWH)	125,626	2,564,753	20,345,011
19. Unit Service Factor	22.6	70.9	68.0
20. Unit Availability Factor	22.6	70.9	68.9
21. Unit Capacity Factor (MDC Net)	20.2	68.0	62.5
22. Unit Capacity Factor (DER Net)	20.1	67.5	61.3
23. Unit Forced Outage Rate	77.4	29.1	24.4

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUELING, AUGUST 2, 1980, 9 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWN/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
9	05/08/80	F	557.0	D	4				CONTINUATION FROM PREVIOUS MONTHS OUTAGE FOR SEISMIC REEVALUATION AND MODIFICATION OF PIPE SUPPORTS PER I&C BULLETIN 79-02.

SUMMARY

THE UNIT CONTINUED OUTAGE FROM 05/08/80 TO 06/23/80 FOR SEISMIC MODIFICATIONS. ON 06/24/80, IT WAS RETURNED TO LINE AND OPERATED NORMALLY FOR THE REST OF THE MONTH.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

MONTICELLO

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Minnesota/Wright 30 miles NW of Minneapolis
2.	Electric Reliability Council	Mid-Continent Area Reliability Coordination Agreement
3.	Type of Reactor	BWR
4.	Power Level (MWT)	1670
5.	Design Electrical Rating (MWE Net)	545
6.	Date of Initial Criticality	December 10, 1970
7.	Date of Commercial Operation	June 30, 1971
8.	Condenser Cooling Method	Once Through with Cooling Tower
9.	Condenser Cooling Water Source	Mississippi River

Utility & Contractor Information

10.	Licensee	Northern States Power Co.
11.	Corporate Address	414 Nicollet Mall - 8th Floor Minneapolis, Minnesota 55401
12.	Corporate Contact	G. H. Neils (Oper.) L. O. Mayer (Lic.)
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	GE
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. F. HEISHMAN
18.	IE Section Chief	W. S. Little
19.	IE Resident Inspector	C. Brown
20.	Licensing Project Manager	K. Eccleston
21.	Docket Number	50-263
22.	License Number/Date of Issue	DPR-22 issued 9/8/70
23.	Public Document Room	Environmental Conservation Library Minneapolis Public Library 300 Nicollet Mall Minneapolis, Minnesota 55401

INSPECTION STATUS

ENFORCEMENT STATUS

AS REQUIRED BY 10 CFR 71.5, NO LICENSEE SHALL TRANSPORT ANY LICENSED MATERIAL OUTSIDE THE CONFINES OF HIS PLANT OR OTHER PLACE OF USE, UNLESS THE LICENSEE COMPLIES WITH THE APPLICABLE REQUIREMENTS OF THE REGULATIONS APPROPRIATE TO THE MODE OF TRANSPORT OF THE DEPARTMENT OF TRANSPORTATION IN 49 CFR PARTS 170-189. CONTRARY TO THE ABOVE, ON MARCH 3, 1980, THE DAY OF THE INSPECTION A CASK SHIPPED FROM YOUR MONTICELLO FACILITY WAS NOT MARKED "RADIOACTIVE LSA" AS REQUIRED BY 49 CFR 173.392(C)(8). THIS A SEVERITY LEVEL IV NONCOMPLIANCE. (8008 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

MANAGERIAL ITEMS

- Mr. L. R. Eliason, former Plant Manager, has been promoted to General Manager, Nuclear Plants. Mr. G. H. Neils is now General Manager, Headquarters Nuclear Group. Mr. W. A. Shamla has been selected as the new Plant Manager. (80-09)

PLANT STATUS

- Routine operation at 100% at months end.

- LAST IE SITE INSPECTION DATE: May 20-22, 1980

- INSPECTION REPORT NO: 80-10

SUMMARY

INSPECTION SUMMARY

- Inspection on March 24, 27-30; April 24-25 and May 6-7, (80-06): Routine, announced inspection of containment integrated leak rate test; local leak rate testing; and followup on previous inspection findings related to leak rate testing. The inspection involved 82 inspector hours onsite by three NRC inspectors. No items of noncompliance or deviations were identified.

- Inspection on April 28 through May 2, (80-09): Routine, unannounced inspection of plant operations, torus-drywell vacuum breaker modification, and recirculation pump seal failures. The inspection involved 66 inspector hours onsite by two NRC inspectors. No items of noncompliance or deviations were identified.

- Inspection on May 20-22, (80-10): A routine, unannounced inspection of fire protection/prevention, IE Bulletin followup, calibration, and review of plant operations. The inspection involved 22 inspector hours onsite by one NRC inspector. No items of noncompliance or deviations were identified in these areas.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

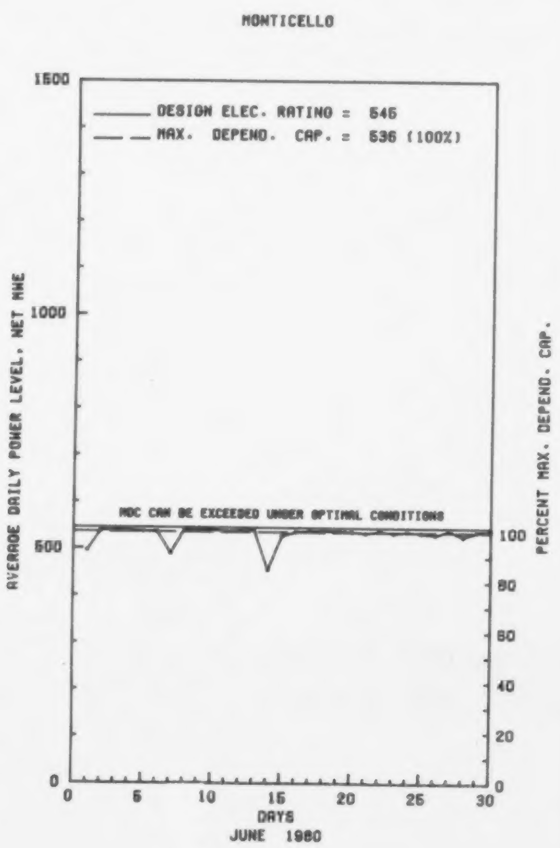
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-20/03L-0	04/27/80	05/27/80	Recirculation valve motor failure.
80-21/03L-0	05/15/80	06/12/80	RCIC control problem.

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 8-4-80

REVIEWED BY NRR DL: Kenneth T. Eccleston **DATE:** 8-5-80

MONTICELLO

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-263
2. Utility Contact: A. L. Myrabo (612) 295-5151
3. Licensed Thermal Power (MWt): 1670
4. Nameplate Rating (Gross MWe): 632 x 0.9 = 569
5. Design Electrical Rating (Net MWe): 545
6. Maximum Dependable Capacity (Gross MWe): 564
7. Maximum Dependable Capacity (Net MWe): 536
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	4,367.0	78,912.0
12. Hours Reactor Was Critical	720.0	2,770.9	64,301.8
13. Reactor Reserve Shutdown Hours	.0	.0	940.7
14. Hours Generator On-Line	720.0	2,690.9	62,661.0
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,188,926	3,903,110	102,845,972
17. Gross Electrical Energy (MWH)	401,620	1,316,230	32,294,290
18. Net Electrical Energy (MWH)	382,372	1,251,576	30,883,780
19. Unit Service Factor	100.0	61.6	79.4
20. Unit Availability Factor	100.0	61.6	79.4
21. Unit Capacity Factor (MDC Net)	99.1	53.5	73.0
22. Unit Capacity Factor (DER Net)	97.6	52.6	71.8
23. Unit Forced Outage Rate	.0	16.9	7.0

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): MARCH 28, 1981 - REFUELING OUTAGE - 56 DAYS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
NONE									

SUMMARY

THE UNIT RAN AT 100% POWER WITH BRIEF REDUCTIONS ON 06/07/80 AND 06/14/80 FOR TESTING.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

NINE MILE POINT 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	New York/Oswego 8 miles NE of Oswego
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	1850
5.	Design Electrical Rating (MWE Net)	620
6.	Date of Initial Criticality	September 5, 1969
7.	Date of Commercial Operation	December 1969
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Lake Ontario

Utility & Contractor Information

10.	Licensee	Niagara Mohawk Power Corporation
11.	Corporate Address	300 Erie Boulevard West Syracuse, New York 13202
12.	Corporate Contact	R. Schneider, Vice President Electric Operations
13.	Architect/Engineer	Utility
14.	Nuclear Steam Supply System	GE
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Section Chief	H. B. KISTER
19.	IE Resident Inspector	W. BAUNACK (X5253)
20.	Licensing Project Manager	P. Polk
21.	Docket Number	50-220
22.	License Number/Date of Issue	DPR-63, 12/26/74
23.	Public Document Room	State University College of Oswego Penfield Library - Documents Oswego, NY 13126 (315)341-2323

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO TS 6.5.1.6, 6.5.1.7 & 6.6.1, THE SITE OPERATIONS REVIEW COMMITTEE FAILED TO DOCUMENT PERFORMANCE OF REQUIRED REVIEWS. (8004 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS:

- The unit operated continuously at essentially full power on four recirculation pumps with the exception of one shutdown to perform Technical Specification core spray isolation valve surveillance. Power limitations have been necessary due to condenser delta temperature limits. Representative stack and offgas release rates are 15,000 and 20 microcuries per second, respectively.

- LAST IE SITE INSPECTION DATE: 6/27/80

- INSPECTION REPORT NO: 50-220/80-06

SUMMARY

INSPECTION SUMMARY

- 50-220/80-04 - May 5-9: Routine, unannounced inspection by two regional based inspectors (62 hours) of onsite review committee performance; site organization and administration; licensee event report followup, and facility tours. One item of non-compliance was found: Failure to document Site Operations Review Committee Reviews.

REPORTS RECEIVED FROM LICENSEE

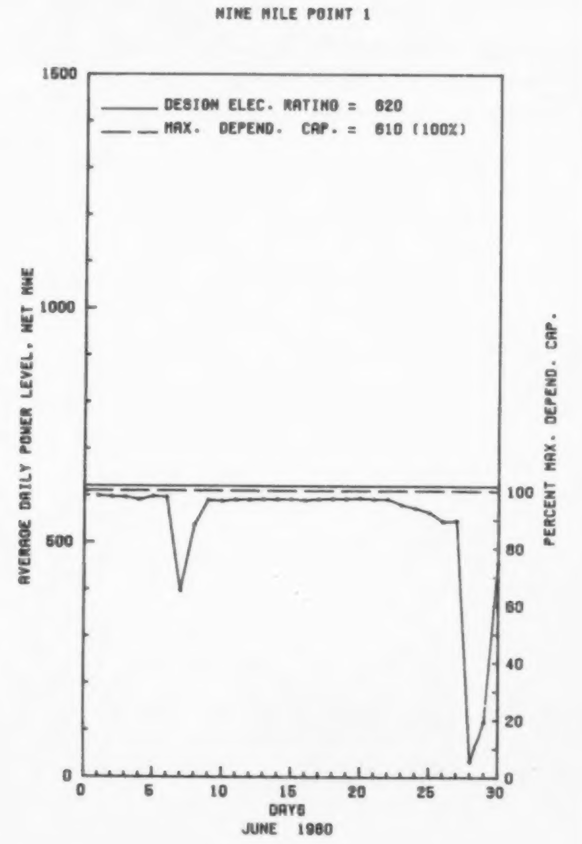
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-009/ 03L	05/21/80	06/03/80	Containment spray isolation valve inoperable
80-010/ 03L	05/28/80	06/03/80	Cardox tank pressure below Technical Specification limit

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 8-4-80

REVIEWED BY NRR DL: Philip J. Polk **DATE:** 8/5/80

NINE MILE POINT 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-220
2. Utility Contact: B. Hawksley (315) 343-2110 x1312
3. Licensed Thermal Power (MWt): 1850
4. Nameplate Rating (Gross MWe): 755 x 0.85 = 642
5. Design Electrical Rating (Net MWe): 620
6. Maximum Dependable Capacity (Gross MWe): 630
7. Maximum Dependable Capacity (Net MWe): 610
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	4,367.0	93,479.0
12. Hours Reactor Was Critical	702.0	4,170.4	69,523.1
13. Reactor Reserve Shutdown Hours	.0	.0	1,204.2
14. Hours Generator On-Line	685.1	4,123.4	66,933.9
15. Unit Reserve Shutdown Hours	.0	.0	20.2
16. Gross Thermal Energy (MWH)	1,217,642	7,204,395	108,945,794
17. Gross Electrical Energy (MWH)	401,683	2,417,598	35,927,108
18. Net Electrical Energy (MWH)	389,042	2,340,025	34,789,790
19. Unit Service Factor	95.2	94.4	71.6
20. Unit Availability Factor	95.2	94.4	71.6
21. Unit Capacity Factor (MDC Net)	88.6	87.8	61.0
22. Unit Capacity Factor (DER Net)	87.2	86.4	60.0
23. Unit Forced Outage Rate	.0	.0	8.8

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-15	06/07/80	S	.0	B	5				LOAD REDUCTION TO 40% POWER TO REPAIR #13 FW PUMP AND ADJUST ROD PATTERN.
80-15	06/28/80	S	34.9	B	1				SHUTDOWN TO PERFORM QUARTERLY CORE SPRAY I.V. TEST AND INSPECT CONDENSER WATER BOXES.

SUMMARY

THE UNIT RAN NORMALLY UNTIL 06/28/80 WITH A POWER REDUCTION OF 40% ON 06/07/80 FOR FW PUMP AND ROD PATTERN ADJUSTMENT. ON 06/28/80, IT WAS SHUTDOWN FOR QUARTERLY INSPECTION.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

NORTH ANNA 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Louisa County, Virginia 40 Miles NW of Richmond, Virginia
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	PWR
4.	Licensee Power Level (MWT)	2775
5.	Design Electrical Rating (MWE Net)	907
6.	Date of Initial Criticality	April 5, 1978
7.	Date of Commercial Operation	June 6, 1978
8.	Condenser Cooling Method	Once through
9.	Condenser Cooling Water Source	Lake Anna

Utility & Contractor Information

10.	Licensee	Virginia Electric & Power Co.
11.	Corporate Address	P. O. Box 26666 Richmond, Virginia 23261
12.	Corporate Contact	J. H. Ferguson Exec. V.P. - Power
13.	Architect/Engineer	Stone & Webster
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. Lewis
18.	IE Section Chief	P. J. Kellog
19.	IE Resident Inspector	E. H. Webster (703) 894-5421
20.	Licensing Project Manager	L. Engle
21.	Docket Number	50-338
22.	License Number/Date of Issue	NPF-4, 11/26/77
23.	Public Document Room	Alderman Library/Manuscripts Dept. Univ. of Virginia/Charlottesville VA 22901 & Louisa County Courthouse, Louisa, VA 23093

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO TECHNICAL SPECIFICATION 4.8.2.A AND 4.8.2.3.2.A, PERIODIC TEST PROCEDURE 1-PT-85 (D.C. DISTRIBUTION SYSTEMS) WAS PERFORMED IN EXCESS OF THE 7-DAY INTERVAL PLUS 25%.
(8022 INFRACTION)

AS REQUIRED BY APPENDIX B, ENVIRONMENTAL TECHNICAL SPECIFICATION 5.5.1, DETAILED WRITTEN PROCEDURES SHALL BE PREPARED AND FOLLOWED FOR ALL ACTIVITIES IN CARRYING OUT ENVIRONMENTAL TECHNICAL SPECIFICATIONS AND THE PROCEDURES SHALL INCLUDE SAMPLING. PROCEDURE H.P. - REMM-4, DATED SEPTEMBER 27, 1979, FOR MILK SAMPLING WAS INADEQUATE IN THAT THE SPECIFIED SAMPLE PRESERVATION TECHNIQUE WAS NOT COMPATIBLE WITH THE ANALYTICAL PROCEDURE USED FOR RADIOIODINE ANALYSIS OF MILK SAMPLES.
(8025 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

UNIT IS OPERATING AT 98% POWER.

LAST IE SITE INSPECTION DATE: MAY 21, 1980

INSPECTION REPORT NO: 50-338/80-26

SUMMARY

INSPECTION SUMMARY

INSPECTION MAY 12-16 (80-22): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 29 INSPECTOR-HOURS ON SITE IN THE INSPECTION OF PREVIOUSLY IDENTIFIED ITEMS. OF THE AREAS INSPECTED, ONE APPARENT ITEM OF NONCOMPLIANCE WAS FOUND IN THE AREA OF PERIODIC TEST PROCEDURES (INFRACTION - FAILURE TO IMPLEMENT SURVEILLANCE TEST).

INSPECTION MAY 12-15 (80-23): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 29 INSPECTOR-HOURS ON SITE IN THE AREAS OF IE BULLETIN 79-14, "SEISMIC ANALYSIS FOR AS-BUILT SAFETY-RELATED PIPING SYSTEMS" AND LICENSEE ACTIONS ON LICENSEE IDENTIFIED ITEMS. OF THE TWO AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION MAY 19-23 - JUNE 2 (80-25): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 16.5 INSPECTOR-HOURS ON SITE IN THE AREAS OF QUALITY CONTROL AND CONFIRMATORY MEASUREMENTS INCLUDING: REVIEW OF THE LABORATORY QUALITY CONTROL PROGRAM; REVIEW OF CHEMICAL AND RADIOCHEMICAL PROCEDURES; QUALITY CONTROL AUDITS; COMPARISON OF THE RESULTS OF SPLIT SAMPLES ANALYZED BY THE LICENSEE AND THE NRC REGION II MOBILE LABORATORY; AND ENVIRONMENTAL MONITORING. OF THE FIVE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN FOUR AREAS; ONE ITEM OF NONCOMPLIANCE WAS FOUND IN ONE AREA (INFRACTION - FAILURE TO HAVE AN ADEQUATE PROCEDURE FOR PRESERVATION OF MILK SAMPLES FOR RADIOIODINE ANALYSIS).

REPORTS RECEIVED FROM LICENSEE

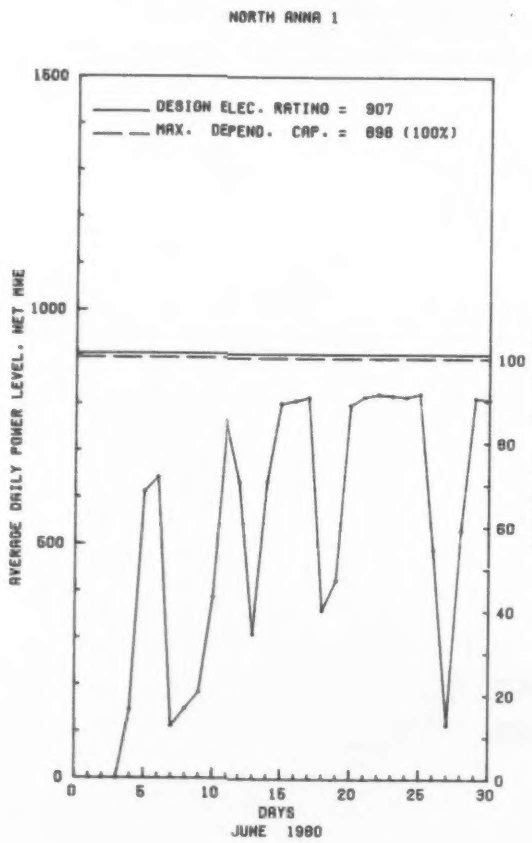
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-040/ 03L-0	04/23/80	05/22/80	CASING COOLING TANK LEVEL INDICATOR OUT OF CALIBRATION DUE TO BAD ZENIER DIODE IN POWER SUPPLY.
80-041/ 03L-0	05/03/80	05/30/80	QUARTERLY SURVEILLANCE TEST OF 3 LOW HEAD SAFETY INJECTION CHECK VALVES PERFORMED LATE.
80-042/ 01T-0	05/09/80	05/22/80	POTENTIAL DAMAGE TO THE CHARGING PUMPS CAUSED BY INADEQUATE FLOW SUBSEQUENT TO SECONDARY SIDE HIGH ENERGY PIPE BREAK.
80-044/ 03L-0	05/10/80	06/09/80	CONTROL FUSES IN CTMT RAD MONITOR BLEW WHEN SAMPLE PUMP WAS RESTARTED.
80-047/ 03L-0	05/10/80	06/09/80	PERMISSIVE P-11 FAILED DURING SURVEILLANCE TEST DUE TO FAULTY LOGIC CARD.

REVIEWED BY OMPA LOEB: ________ **DATE:** 8/4/80

REVIEWED BY NRR DL: LBEngle **DATE:** 8/5/80

NORTH ANNA 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-338
2. Utility Contact: W. R. Madison (703) 894-5151
3. Licensed Thermal Power (MWt): 2775
4. Nameplate Rating (Gross MWe): 1089 x 0.9 = 980
5. Design Electrical Rating (Net MWe): 907
6. Maximum Dependable Capacity (Gross MWe): 928
7. Maximum Dependable Capacity (Net MWe): 898
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	4,367.0	18,144.0
12. Hours Reactor Was Critical	634.8	3,434.6	13,655.8
13. Reactor Reserve Shutdown Hours	10.9	52.8	201.1
14. Hours Generator On-Line	598.9	3,301.9	13,355.0
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,295,756	8,163,833	34,443,746
17. Gross Electrical Energy (MWH)	393,487	2,548,338	10,894,637
18. Net Electrical Energy (MWH)	370,683	2,392,496	10,245,942
19. Unit Service Factor	83.2	75.6	73.6
20. Unit Availability Factor	83.2	75.6	73.6
21. Unit Capacity Factor (MDC Net)	57.3	61.0	62.9
22. Unit Capacity Factor (DER Net)	56.8	60.4	62.3
23. Unit Forced Outage Rate	16.8	13.9	11.4

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUELING; SIX WEEKS; DECEMBER 1980 AND JANUARY, 1981.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-13	05/22/80	F	88.0	B	4		SF	GENERA	HI-HI S/G WATER LEVEL. FEED VALVE NEEDED REPAIR.
80-14	06/06/80	S	87.7	H	5				REDUCED TURBINE POWER TO 200 MWE FOR STEAM GENERATOR SECONDARY SIDE BORIC ACID SOAK.
80-15	06/12/80	F	8.1	G	3				REACTOR TRIP DUE TO OPERATIONAL ERROR IN OPERATING WRONG SWITCH IN PROTECTION AND CONTROL RACK WHILE PERFORMING PERIODIC TEST.
80-16	06/14/80	F	6.0	A	5				REDUCED REACTOR POWER TO 40% TO ALLOW CONTAINMENT ENTRY TO REPAIR LT-1474 (A S/G LEVEL) ROOT VALVE. REPAIR VALVE AND ESCALATED TO 100% POWER.
80-17	06/18/80	F	6.4	A	3				REACTOR TRIP DUE TO A LOOSE JUMPER IN PROCESS RACK NO. 6. JUMPER REPAIRED AND REACTOR RESTARTED.
80-18	06/26/80	F	15.9	H	3				OVERTEMPERATURE DELTA T REACTOR TRIP WHILE CALIBRATING N43 RECEIVED A SPURIOUS SPIKE ON ONE OF THE OTHER TWO CHANNELS. A FAULTY CARD WAS CHANGED OUT IN CHANNEL II AND TEST RECORDER INSTALLED.
80-19	06/27/80	F	2.7	A	1				REMOVED TURBINE GENERATOR FROM SERVICE DUE TO B MAIN STEAM TRIP VALVE AIR LINE BROKEN. REPAIRED LINE AND PLACED TURBINE GENERATOR BACK IN SERVICE.

SUMMARY

THE PLANT RETURNED TO SERVICE AFTER STEAM GENERATOR VALVE REPAIR ON 06/04/80. ON 06/06/80 LOAD WAS REDUCED TO MINIMIZE STEAM GENERATOR TUBE DENTING; ON 06/14/80 FOR STEAM GENERATOR ROOT VALVE REPAIR. OUTAGES OCCURRED ON 06/12/80 DUE TO PERSONNEL ERROR. ON 06/18/80 DUE TO JARRING A LOOSE JUMPER ON A PROCESS CARD, ON 06/26/80 DUE TO SPURIOUS INSTRUMENT SPIKE CAUSING OVERTEMPERATURE TRIP AND ON 06/27/80 FOR REPAIR OF A BROKEN AIR LINE ON THE MAIN STEAM TRIP VALVE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

OCONEE 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	South Carolina/Oconee 30 miles W of Greenville
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2568
5.	Design Electrical Rating (MWE Net)	887
6.	Date of Initial Criticality	April 19, 1973
7.	Date of Commercial Operation	July 15, 1973
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Keowee Lake

Utility & Contractor Information

10.	Licensee	Duke Power Company
11.	Corporate Address	422 S. Church Street Charlotte, North Carolina 28242
12.	Corporate Contact	W. O. Parker, Jr.
13.	Architect/Engineer	DPC and Bechtel (Containment Bldg.)
14.	Nuclear Steam Supply System	B&W
15.	Constructor	DPC

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R.C. Lewis
18.	IE Section Chief	R. MARTIN
19.	IE Resident Inspector	F. Jape (803) 882-6927
20.	Licensing Project Manager	M. B. Fairtile
21.	Docket Number	50-269
22.	License Number/Date of Issue	DPR-38, 2/6/73
23.	Public Document Room	Oconee County Library 201 S. Spring Street Walhalla, South Carolina 29691

INSPECTION STATUS

ENFORCEMENT STATUS

AS REQUIRED BY 10 CFR 20.203(C)(2) EACH ENTRANCE OR ACCESS POINT TO A HIGH RADIATION AREA SHALL BE MAINTAINED LOCKED EXCEPT DURING PERIODS WHEN ACCESS TO THE AREA IS REQUIRED, WITH POSITIVE CONTROL OVER INDIVIDUAL ENTRY. CONTRARY TO THE ABOVE, ON OCTOBER 11, 1979 THE DOOR LEADING TO ROOM 251 (DEBORATING DEMINERALIZER HATCH AREA) WAS FOUND UNLOCKED. POSITIVE CONTROL OVER EACH INDIVIDUAL ENTRY WAS NOT IN EFFECT.
(7925 INFRACTION)

CONTRARY TO TECHNICAL SPECIFICATION 6.4.1, ON MAY 22, 1980 FUEL HANDLING PROCEDURE OP/0/A/1506/01 WAS USED TO TRANSFER A SPENT FUEL ASSEMBLY TO THE SHIPPING CASK. THIS PROCEDURE WAS NOT CHECKED WITH THE CONTROL COPY PRIOR TO USE AND, IN FACT HAD BEEN REVISED BY TWO REVISIONS DATED DECEMBER 1, 1979 AND MAY 13, 1980.
(8020 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

UNIT OPERATING ON THREE REACTOR COOLANT PUMPS, LIMITING POWER TO 75%.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

+ SHUT DOWN.

LAST IE SITE INSPECTION DATE: MAY 1-30, 1980

INSPECTION REPORT NO: 50-269/80-20

SUMMARY

INSPECTION SUMMARY

+ INSPECTION MAY 1-30 (80-20): THIS INSPECTION INVOLVED 83.3 INSPECTOR-HOURS ON SITE IN THE AREAS OF PLANT OPERATIONS, PHYSICAL PROTECTION, OFF-SITE FUEL SHIPMENT AND SURVEILLANCE ACTIVITIES. OF THE FOUR AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN THREE AREAS; ONE APPARENT ITEM OF NONCOMPLIANCE WAS FOUND IN ONE AREA (INFRACTION - USING SUPERCEDED PROCEDURE TO OPERATE FUEL HANDLING EQUIPMENT).

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

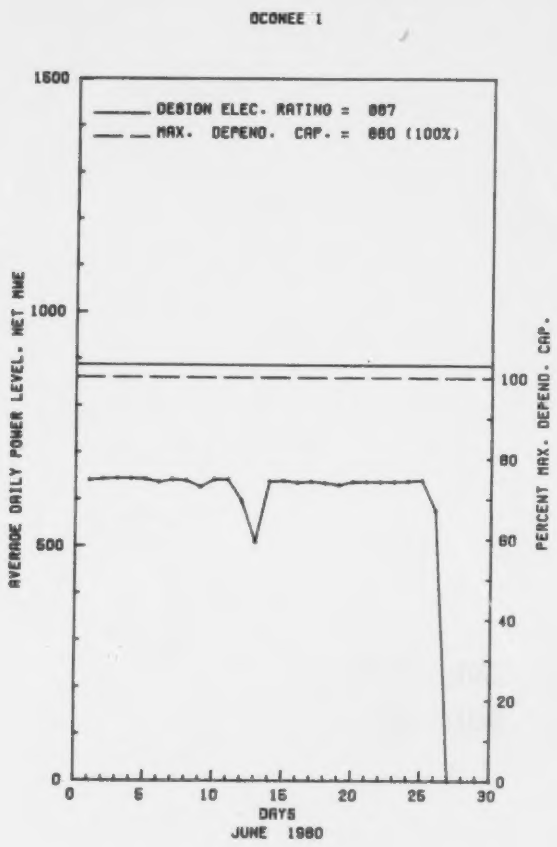
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-010	04/16/80	05/12/80	RB PERSONNEL HATCH INNER DOOR LEAK DUE TO FAILURE OF GASKET TO SEAL.
80-012	05/01/80	05/29/80	TURBINE DRIVEN EMERGENCY FEEDWATER PUMP ISOLATED FROM STEAM SUPPLY, S/G 1B.
80-013	05/09/80	06/05/80	TURBINE DRIVEN EM FEEDWATER PUMP INOP DUE TO FAILED INBOARD BEARING.
80-016	05/23/80	05/29/80	4 CRACKED FUEL ASSEMBLY HOLD DOWN SPRINGS FOUND ON SPENT FUEL IN UNIT 3 POOL. FUEL IS FROM UNITS 1 AND 3.

REVIEWED BY OMPA LOEB: ______ **DATE:** 8/4/80

REVIEWED BY NRR DL: ______ **DATE:** 8/5/80

OCONEE 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-269
2. Utility Contact: J. A. Reavis (704) 373-8552
3. Licensed Thermal Power (MWt): 2568
4. Nameplate Rating (Gross MWe): 1038 x 0.9 = 934
5. Design Electrical Rating (Net MWe): 887
6. Maximum Dependable Capacity (Gross MWe): 899
7. Maximum Dependable Capacity (Net MWe): 860

CAPACITY FACTORS CALCULATED WITH A WEIGHTED AVERAGE

8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	4,367.0	61,208.0
12. Hours Reactor Was Critical	629.5	2,951.0	43,465.7
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	626.4	2,883.2	40,830.3
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,225,907	6,004,677	95,222,796
17. Gross Electrical Energy (MWH)	412,850	2,134,640	33,048,910
18. Net Electrical Energy (MWH)	392,432	2,025,498	31,256,997
19. Unit Service Factor	87.0	66.0	66.9
20. Unit Availability Factor	87.0	66.0	66.9
21. Unit Capacity Factor (MDC Net)	63.4	53.9	58.3
22. Unit Capacity Factor (DER Net)	61.4	52.3	57.8
23. Unit Forced Outage Rate	.0	10.1	17.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
TMI RELATED MODIFICATION - NOVEMBER 1, 1980 - 4 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: 07/10/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
6P	06/01/80	F	.0	A	5		CB	MOTORX	1B1 RCP REMAINS OUT OF SERVICE. WILL REQUIRE OUTAGE FOR INSPECTION BEFORE RESTARTING.
4	06/27/80	S	93.6	D	1		ZZ	ZZZZZZ	NRC REQUIRED MODIFICATIONS OF EMERGENCY POWER SUPPLY NSM 1531. ALSO INSPECTION OF 1B1 RCP LOWER MOTOR BEARING.

SUMMARY

THE UNIT BEGAN THE MONTH AT REDUCED POWER DUE TO 1B1 RCP BEING OUT OF SERVICE. ON 06/27/80, THE UNIT BEGAN AN OUTAGE FOR MODIFICATIONS TO EMERGENCY POWER SYSTEMS.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

OCONEE 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1. Location (state/county) (nearest population center)	South Carolina/Oconee 30 miles W of Greenville
2. Electric Reliability Council	Southeastern Electric Reliability Council
3. Type of Reactor	PWR
4. Power Level (MWT)	2568
5. Design Electrical Rating (MWE Net)	887
6. Date of Initial Criticality	November 11, 1973
7. Date of Commercial Operation	September 9, 1974
8. Condenser Cooling Method	Once Through
9. Condenser Cooling Water Source	Keowee Lake

Utility & Contractor Information

10. Licensee	Duke Power Company
11. Corporate Address	422 S. Church Street Charlotte, North Carolina 28242
12. Corporate Contact	W. O. Parker, Jr.
13. Architect/Engineer	DPC/Bechtel
14. Nuclear Steam Supply System	B&W
15. Constructor	DPC

Regulatory Information

16. IE Region responsible	II
17. IE Branch Chief	R.C. Lewis
18. IE Section Chief	R. MARTIN
19. IE Resident Inspector	F. Jape (803) 882-6927
20. Licensing Project Manager	M. B. Fairtile
21. Docket Number	50-270
22. License Number/Date of Issue	DPR-47, 10/6/73
23. Public Document Room	Oconee County Library 201 S. Spring Street Walhalla, South Carolina 29691

INSPECTION STATUS

ENFORCEMENT STATUS

AS REQUIRED BY TECHNICAL SPECIFICATION 6.6.2.1.B, WRITTEN REPORTS OF THE TYPES OF EVENTS LISTED IN THAT SPECIFICATION ARE TO BE SUBMITTED TO THE DIRECTOR, OFFICE OF INSPECTION AND ENFORCEMENT, REGION II, WITHIN 30 DAYS OF THE OCCURRENCE OF THE EVENT. CONTRARY TO THE ABOVE, FOUR OF THE THIRTEEN LICENSEE EVENTS REPORTS FOR UNITS 1 AND 3 SUBMITTED DURING THE PERIOD NOVEMBER 1, 1978 TO JANUARY 15, 1979 WERE SUBMITTED FROM ONE TO THREE DAYS LATER THAN REQUIRED.
(7901 DEFICIENCY)

AS REQUIRED BY 10 CFR 20.203(C)(2) EACH ENTRANCE OR ACCESS POINT TO A HIGH RADIATION AREA SHALL BE MAINTAINED LOCKED EXCEPT DURING PERIODS WHEN ACCESS TO THE AREA IS REQUIRED, WITH POSITIVE CONTROL OVER INDIVIDUAL ENTRY. CONTRARY TO THE ABOVE, ON OCTOBER 11, 1979 THE DOOR LEADING TO ROOM 251 (DEBORATING DEMINERALIZER HATCH AREA) WAS FOUND UNLOCKED. POSITIVE CONTROL OVER EACH INDIVIDUAL ENTRY WAS NOT IN EFFECT.
(7923 INFRACTION)

AS REQUIRED BY 10CFR50 APPENDIX B, CRITERIA XIV AND AS IMPLEMENTED IN SECTION 17.2.14 OF DUKE-1-A, QUALITY ASSURANCE PROGRAM MEASURES TO MAINTAIN EQUIPMENT STATUS SHALL BE UTILIZED. THE ADMINISTRATIVE POLICY MANUAL FOR NUCLEAR STATIONS, SECTION 3.1.3 FURTHER REQUIRES MEASURES TO BE ESTABLISHED TO ASSURE ACCURATE KNOWLEDGE OF COMPONENT STATUS IS AVAILABLE FOR APPROPRIATE ACTION CONSISTENT WITH THE STATUS OF THE ITEM. THE BORON CONTENT OF A DEMINERALIZER PLACED INTO SERVICE ON MAY 5, 1980, WAS NOT ACCURATELY REFLECTED IN THE STATUS BOARD IN THE CONTROL ROOM. CONSEQUENTLY AN UNPLANNED REACTIVITY ADDITION OF APPROXIMATELY 4.45% DELTA K/K OCCURRED.
(8015 INFRACTION)

AS REQUIRED BY TECHNICAL SPECIFICATION 6.4.1 A AND B DETAILED PROCEDURES FOR CONTROL AND OPERATION OF SYSTEMS AND COMPONENTS INVOLVING NUCLEAR SAFETY OF THE FACILITY SHALL BE PROVIDED. THIS REQUIREMENT IS ALSO CONTAINED IN DUKE-1-A, QUALITY ASSURANCE PROGRAM AND IS FURTHER IMPLEMENTED IN SECTION 3.1.4 OF THE ADMINISTRATIVE POLICY MANUAL FOR NUCLEAR STATIONS. FOLLOWING REFUELING ACTIVITIES ON OR ABOUT APRIL 11, 1980 DATA WAS NOT AVAILABLE OR PROVIDED TO CONTROL ROOM OPERATORS FOR USE IN OP/2/A/1103/15, "REACTIVITY BALANCE". CONSEQUENTLY THE CONTROL ROOM OPERATORS COULD NOT CALCULATE SHUTDOWN MARGIN AS REQUIRED BY TECHNICAL SPECIFICATION 3.1.3.4.
(8015 INFRACTION)

AS REQUIRED BY TECHNICAL SPECIFICATION 6.2.1, WHICH IS IMPLEMENTED BY OCONEE NUCLEAR STATION DIRECTIVE 3.1.10, ANY REPORTABLE OCCURRENCE SHALL BE INVESTIGATED PROMPTLY. ON JUNE 7, 1980 A REPORTABLE EVENT INVOLVING A REACTIVITY INSERTION OF MORE THAN 0.5% DELTA K/K OCCURRED BUT WAS NOT PROMPTLY INVESTIGATED.
(8015 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANT AND PROCEDURES):

ANTICIPATED CRITICALITY FOLLOWING OUTAGE 6/16/80.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

Unit shutdown most of month.

LAST IE SITE INSPECTION DATE: MAY 28-30, 1980

INSPECTION REPORT NO: 50-270/80-16

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-002/03L/0	-	05/23/80	HYDRAULIC SHOCK SUPPRESSOR DECLARED INOPERATIVE.
80-003/03L-0	05/07/80	05/21/80	SPARE PURIFICATION DEMINERALIZER USE RESULTED IN DEBORATION FROM 1895 TO 1539 PPM. UNPLANNED REACTIVITY INSERTION ON GREATER THAN 0.5% DK/K. UNIT IN REFUELING OUTAGE.

SUMMARY

INSPECTION SUMMARY

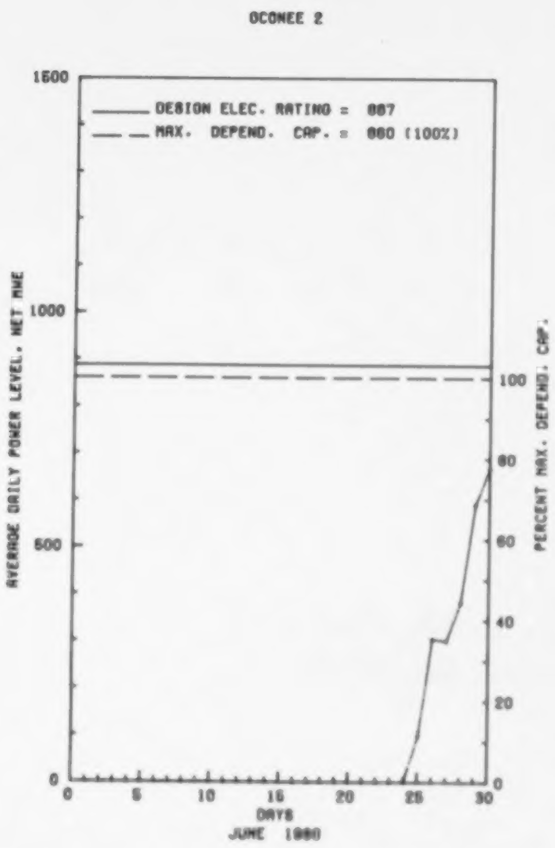
INSPECTION MAY 1-30 (80-15): THIS INSPECTION INVOLVED 83.3 INSPECTOR-HOURS ON SITE IN THE AREAS OF PLANT OPERATIONS, PHYSICAL PROTECTION, OFFSITE FUEL SHIPMENT AND SURVEILLANCE ACTIVITIES. OF THE FOUR AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN THREE AREAS; THREE APPARENT ITEMS OF NONCOMPLIANCE WERE FOUND IN ONE AREA (INFRACTION - FAILURE TO MAINTAIN CORRECT STATUS OF DEMINERALIZER; INFRACTION - FAILURE TO PROVIDE DATA TO DETERMINE A SHUTDOWN MARGIN; INFRACTION - FAILURE TO PROMPTLY INVESTIGATE AN UNPLANNED REACTIVITY INSERTION OF MORE THAN 0.5% DK/K).

INSPECTION MAY 28-30 (80-16): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 72 INSPECTOR-HOURS ON SITE IN THE AREAS OF CONTAINMENT INTEGRATED LEAK RATE TESTING WHICH INCLUDED INSPECTION OF THE TEST PROCEDURE, INSTRUMENT CALIBRATION, DATA PROCESSING AND VALVE ALIGNMENT. OF THE FOUR AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED IN THREE AREAS; TWO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED IN ONE AREA (INFRACTION - FAILURE TO FOLLOW PROCEDURE; AND INFRACTION - FAILURE TO PROVIDE ADEQUATE PROCEDURE).

REVIEWED BY OMPA LOEB: DATE: 8/4/80
REVIEWED BY NRR DL: DATE: 8/5/80

OCONEE 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-270
2. Utility Contact: J. A. Reavis (704) 373-8552
3. Licensed Thermal Power (MWt): 2568
4. Nameplate Rating (Gross MWe): 1038 x 0.9 = 934
5. Design Electrical Rating (Net MWe): 887
6. Maximum Dependable Capacity (Gross MWe): 899
7. Maximum Dependable Capacity (Net MWe): 860

CAPACITY FACTORS CALCULATED WITH A WEIGHTED AVERAGE

8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	4,367.0	50,928.3
12. Hours Reactor Was Critical	219.7	1,756.0	35,352.8
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	134.4	1,656.2	34,432.3
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	189,548	3,515,548	81,562,952
17. Gross Electrical Energy (MWH)	63,860	1,187,899	27,781,556
18. Net Electrical Energy (MWH)	47,955	1,113,269	25,277,827
19. Unit Service Factor	18.7	37.9	67.6
20. Unit Availability Factor	18.7	37.9	67.6
21. Unit Capacity Factor (MDC Net)	7.7	29.6	58.7
22. Unit Capacity Factor (DER Net)	7.5	28.7	56.2
23. Unit Forced Outage Rate	41.6	6.2	19.6

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): TMI RELATED MOD. - SEPT. 6, 1980 - 6 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
2A	03/06/80	S	8.0	B	4		ZZ	PIPEXX	COMPLETION OF PIPE HANGER/SUPPORT INSPECTION AND MODIFICATION (IE BULLETIN 79-02 AND 79-14).
2B	06/01/80	S	398.3	B	4		ZZ	ZZZZZZ	COMPLETED REACTOR BUILDING LEAK RATE TEST.
2C	06/17/80	F	30.1	D	4		ZZ	ZZZZZZ	LOW BORON CONCENTRATION IN "2A" CORE FLOOD TANK.
2D	06/19/80	F	48.4	A	4		CB	VALVEX	LEAKING FLANGE ON 2RC-68 (PRESSURIZER CODE RELIEF VALVE).
2E	06/21/80	S	52.1	B	4		ZZ	ZZZZZZ	ZERO POWER PHYSICS TEST.
2F	06/23/80	S	6.1	B	4		ZZ	ZZZZZZ	POWER ESCALATION TESTING.
2G	06/23/80	S	9.9	B	4		HA	TURBIN	MAIN STEAM STOP VALVE CLOSURE TEST.
2H	06/24/80	S	5.6	B	4		ZZ	ZZZZZZ	POWER ESCALATION TESTING.
2I	06/24/80	S	9.3	B	4		CB	HTEXCH	PERFORM STEAM GENERATOR LEVEL CONTROL ALTERNATE SYSTEMS TUNING.
2J	06/24/80	F	18.0	A	4		CB	VALVEX	PRESSURIZER CODE RELIEF VALVE 2RC-68 FLANGE LEAK REPAIR.
7P	06/25/80	F	.0	A	5		HA	TURBIN	HOLD DUE TO TEMPERATURE DIFFERENTIAL ON TURBINE.
8P	06/26/80	S	.0	B	5		ZZ	ZZZZZZ	HOLD FOR POWER ESCALATION TESTING AT 40%.
9P	06/28/80	F	.0	A	5		CH	PUMPXX	HOLD DUE TO 2B FDWP ECCENTRICITY HIGH.
10P	06/29/80	S	.0	B	5		ZZ	ZZZZZZ	HOLD FOR POWER ESCALATION TESTING AT 75%.
11P	06/30/80	S	.0	B	5		ZZ	ZZZZZZ	HOLD FOR POWER ESCALATION TESTING AT 90%.

SUMMARY

THE UNIT BEGAN THE MONTH IN A REFUELING OUTAGE. ON 06/25/80, THE UNIT WAS BROUGHT BACK TO SERVICE. FOLLOWING POWER ESCALATION TESTING, THE UNIT ENDED THE MONTH AT 94% POWER.

LEGEND

[1] Type:
F-Forced
S-Scheduled

[2] Reason:
A-Equipment Failure
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error
H-Other

[3] Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Continuations
5-Load Reduction
9-Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source

OCONEE 3

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	South Carolina/Oconee 30 miles W of Greenville
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2568
5.	Design Electrical Rating (MWE Net)	887
6.	Date of Initial Criticality	September 5, 1974
7.	Date of Commercial Operation	December 16, 1974
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Keowee Lake

Utility & Contractor Information

10.	Licensee	Duke Power Company
11.	Corporate Address	422 S. Church Street Charlotte, North Carolina 28242
12.	Corporate Contact	W. O. Parker, Jr.
13.	Architect/Engineer	DPC/Bechtel
14.	Nuclear Steam Supply System	B&W
15.	Constructor	DPC

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R.C. Lewis
18.	IE Section Chief	R. MARTIN
19.	IE Resident Inspector	F. Jape (803) 882-6927
20.	Licensing Project Manager	M. S. Fairtile
21.	Docket Number	50-287
22.	License Number/Date of Issue	DPR-55, 7/19/74
23.	Public Document Room	Oconee County Library 201 S. Spring Street Walhalla, South Carolina 29691

INSPECTION STATUS

ENFORCEMENT STATUS

AS REQUIRED BY TECHNICAL SPECIFICATION 6.6.2.1.B, WRITTEN REPORTS OF THE TYPES OF EVENTS LISTED IN THAT SPECIFICATION ARE TO BE SUBMITTED TO THE DIRECTOR, OFFICE OF INSPECTION AND ENFORCEMENT, REGION II, WITHIN 30 DAYS OF THE OCCURRENCE OF THE EVENT. CONTRARY TO THE ABOVE, FOUR OF THE THIRTEEN LICENSEE EVENTS REPORTS FOR UNITS 1 AND 3 SUBMITTED DURING THE PERIOD NOVEMBER 1, 1978 TO JANUARY 15, 1979 WERE SUBMITTED FROM ONE TO THREE DAYS LATER THAN REQUIRED.
(7901 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

PLANT SHUTDOWN FOR REPAIRS TO STEAM GENERATOR.

LAST IE SITE INSPECTION DATE: MAY 1-30, 1980

INSPECTION REPORT NO: 50-287/80-16

SUMMARY

INSPECTION SUMMARY

INSPECTION MAY 1-30 (80-16): THIS INSPECTION INVOLVED 81.3 INSPECTOR HOURS ON SITE IN THE AREAS OF PLANT OPERATIONS, PHYSICAL PROTECTION, OFF SITE FUEL SHIPMENT AND SURVEILLANCE ACTIVITIES. OF THE AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

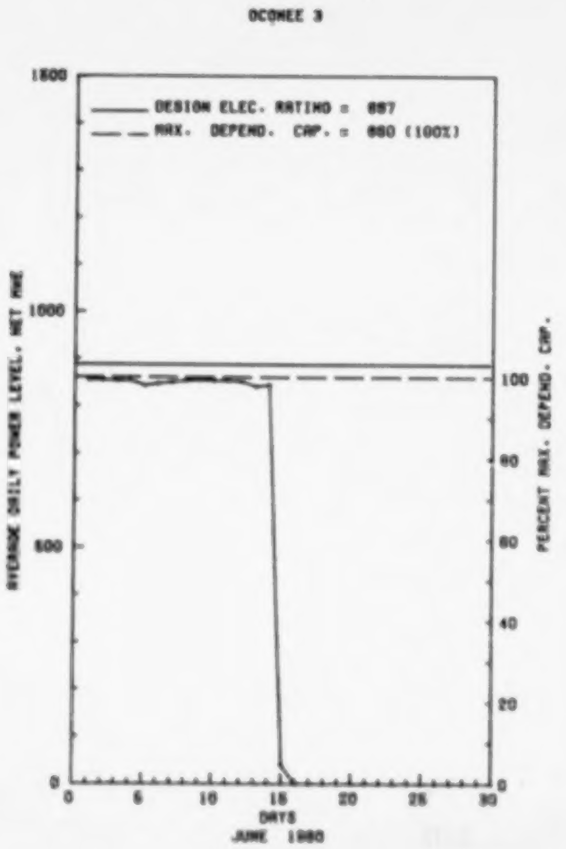
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-006/03L-0	04/23/80	05/23/80	BWST LEVEL BELOW TECHNICAL SPECIFICATION LIMIT DUE TO RBS PUMP B SUCTION CHECK VALVE FAILURE TO RESEAT AFTER TESTING.
80-007/03L-0	04/28/80	05/28/80	GASEOUS WASTE RELEASES MADE WHILE GAS MONITORS CONSIDERED INOPERABLE DUE TO MISSED SURVEILLANCES.
80-008/03L-0	05/01/80	06/06/80	POWER LOST TO RPS CHANNEL B DUE TO INVERTER TRIPS.

REVIEWED BY OMPA LOEB: DATE: 8/4/80

REVIEWED BY NRR DL: DATE: 9/5/80

OCONEE 3

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-287
2. Utility Contact: J. A. Reavis (704) 373-8552
3. Licensed Thermal Power (MWt): 2568
4. Nameplate Rating (Gross MWe): 1038 x 0.9 = 934
5. Design Electrical Rating (Net MWe): 887
6. Maximum Dependable Capacity (Gross MWe): 899
7. Maximum Dependable Capacity (Net MWe): 860
8. If Changes Occur in Capacity Ratings above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

CAPACITY FACTORS CALCULATED WITH A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	4,367.0	48,575.3
12. Hours Reactor Was Critical	366.1	2,877.7	36,771.6
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	339.1	2,825.1	33,287.2
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	858,295	7,172,289	81,553,896
17. Gross Electrical Energy (MWH)	299,570	2,479,660	28,230,986
18. Net Electrical Energy (MWH)	285,486	2,363,118	26,859,676
19. Unit Service Factor	47.1	64.7	69.8
20. Unit Availability Factor	47.1	64.7	69.8
21. Unit Capacity Factor (MDC Net)	45.9	62.9	64.0
22. Unit Capacity Factor (DER Net)	44.5	61.1	62.1
23. Unit Forced Outage Rate	52.9	16.7	10.0

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUEL - DEC. 20, 1980 - 10 WEEKS.

25. If Shut Down At End of Report Period, Estimated Date of Startup: 07/26/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
2	06/15/80	F	380.9	A	1		CB		HTEXCH TUBE LEAK IN "A" STEAM GENERATOR.

SUMMARY

THE UNIT RAN AT NEARLY FULL POWER UNTIL SHUTDOWN ON 06/15/80 FOR REPAIRS IN THE STEAM GENERATOR.

LEGEND

[1] Type:
F–Forced
S–Scheduled

[2] Reason:
A–Equipment Failure
B–Maintenance or Test
C–Refueling
D–Regulatory Restriction
E–Operator Training & License Examination
F–Administrative
G–Operational Error
H–Other

[3] Method:
1–Manual
2–Manual Scram
3–Automatic Scram
4–Continuations
5–Load Reduction
9–Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

OYSTER CREEK

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	New Jersey/Ocean 9 miles S of Toms River
2.	Electric Reliability Council	Mid-Atlantic Area Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	1930
5.	Design Electrical Rating (MWE Net)	650
6.	Date of Initial Criticality	May 3, 1969
7.	Date of Commercial Operation	December 1969
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Barnegat Bay

Utility & Contractor information

10.	Licensee	Jersey Central Power & Light Co.
11.	Corporate Address	Madison Avenue At Punch Bowl Road Morristown, New Jersey 07960
12.	Corporate Contact	I. Finfrock, Vice President Power Generation
13.	Architect/Engineer	Burns & Roe
14.	Nuclear Steam Supply System	GE
15.	Constructor	J. A. Jones

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	IE Section Chief	R. R. KEIMIG
19.	IE Resident Inspector	L. BRIGGS (X5306) (609) 693-0702 NRC RESIDENT OFFICE
20.	Licensing Project Manager	W. A. Paulson
21.	Docket Number	50-219
22.	License Number/Date of Issue	DPR-16, 4/9/69
23.	Public Document Room	Ocean County Library 15 Hooper Avenue Toms River, New Jersey 08753

INSPECTION STATUS

ENFORCEMENT STATUS

10 CFR 20.103(C), "EXPOSURE OF INDIVIDUALS TO CONCENTRATIONS OF RADIOACTIVE MATERIALS IN AIR IN RESTRICTED AREAS" REQUIRES IN PART THAT: WHEN RESPIRATORY PROTECTIVE EQUIPMENT IS USED TO LIMIT THE INHALATION OF AIRBORNE RADIOACTIVE MATERIAL PURSUANT TO PARAGRAPH (B)(2) OF THIS SECTION, THE LICENSEE MAY MAKE ALLOWANCE FOR SUCH USE IN ESTIMATING EXPOSURE OF INDIVIDUALS TO SUCH MATERIALS PROVIDED THAT SUCH EQUIPMENT IS USED AS STIPULATED IN REGULATORY GUIDE 8.15, "ACCEPTABLE PROGRAMS FOR RESPIRATORY PROTECTION." SECTION C.4 OF REGULATORY GUIDE 8.15 STATES IN PART, "THE LICENSEE IS TO MAINTAIN AND IMPLEMENT A RESPIRATORY PROTECTION PROGRAM THAT INCLUDES, AS A MINIMUM...E. WRITTEN OPERATIONAL AND ADMINISTRATIVE PROCEDURES FOR CONTROL, ISSUANCE, PROPER USE, AND RETURN OF RESPIRATORY PROTECTIVE EQUIPMENT..." CONTRARY TO THE ABOVE, AS OF JANUARY 23, 1980, NO ADEQUATE OPERATIONAL OR ADMINISTRATIVE PROCEDURES ADDRESSING CONTROL AND ISSUANCE OF BREATHING AIR FITTINGS WAS INCLUDED IN THE LICENSEE'S RESPIRATORY PROTECTION PROGRAM. LICENSEE PROCEDURE 915.5 ADDRESSED RESPONSIBILITY FOR THIS AREA, HOWEVER, THIS GUIDANCE WAS INADEQUATE TO PREVENT IMPROPER USE OF THE FITTINGS.
(8003 INFRACTION)

10 CFR 20.203(F) REQUIRES THAT EACH CONTAINER OF LICENSED MATERIAL SHALL BEAR A DURABLE, CLEARLY VISIBLE LABEL IDENTIFYING THE RADIOACTIVE CONTENTS. THE LABEL SHALL BEAR THE RADIATION CAUTION SYMBOL AND THE WORDS "CAUTION RADIOACTIVE MATERIAL" OR "DANGER RADIOACTIVE MATERIAL". THE LABEL SHALL ALSO PROVIDE SUFFICIENT INFORMATION TO PERMIT INDIVIDUALS HANDLING OR USING THE CONTAINERS, OR WORKING IN THE VICINITY THEREOF TO TAKE PRECAUTIONS TO AVOID OR MINIMIZE EXPOSURES. 1. CONTRARY TO THE ABOVE, ON JANUARY 21, 1980 TWO 4' X 4' X 6' CONTAINERS LOCATED OUTSIDE THE OLD RADWASTE BUILDING CONTAINED LICENSED RADIOACTIVE MATERIAL AND WERE NOT LABELED AND NO EXEMPTION APPLIED. 2. CONTRARY TO THE ABOVE, ON JANUARY 25, 1980, TWO 55 GALLON DRUMS, LOCATED ON THE FLOOR OF THE TORUS CONTAINED LICENSED RADIOACTIVE MATERIAL AND WERE NOT LABELED AND NO EXEMPTION APPLIED. THE DRUMS INDICATED UP TO 5000 MILLIREM/HR ON CONTACT.
(8003 DEFICIENCY)

10 CFR 20.201, "SURVEYS", STATES IN PARAGRAPH (B), "EACH LICENSEE SHALL MAKE OR CAUSE TO BE MADE SUCH SURVEYS AS MAY BE NECESSARY FOR HIM TO COMPLY WITH THE REGULATIONS IN THIS PART. A SURVEY AS DEFINED IN PARAGRAPH 20.201(A) MEANS, "AN EVALUATION OF THE RADIATION HAZARDS INCIDENT TO THE PRODUCTION, USE, RELEASE, DISPOSAL, OR PRESENCE OF RADIOACTIVE MATERIALS OR OTHER SOURCES OF RADIATION UNDER A SPECIFIC SET OF CONDITIONS. WHEN APPROPRIATE, SUCH EVALUATION INCLUDES A PHYSICAL SURVEY OF THE LOCATION OF MATERIALS AND EQUIPMENT, AND MEASUREMENTS OF LEVELS OF RADIATION OR CONCENTRATIONS OF RADIOACTIVE MATERIAL PRESENT." 10 CFR 20.202, "PERSONNEL MONITORING" STATES IN PARAGRAPH (A) THAT "EACH LICENSEE SHALL SUPPLY APPROPRIATE PERSONNEL MONITORING EQUIPMENT TO, AND SHALL REQUIRE THE USE OF SUCH EQUIPMENT BY...(1) EACH INDIVIDUAL WHO ENTERS A RESTRICTED AREA UNDER SUCH CIRCUMSTANCES THAT HE RECEIVES, OR IS LIKELY TO RECEIVE, A DOSE IN ANY CALENDAR QUARTER IN EXCESS OF 25 PERCENT OF THE A PPLICABLE VALUE SPECIFIED IN PARAGRAPH (A) OF PARA 20.101. CONTRARY TO THE ABOVE AS OF JANUARY 25, 1980 INDIVIDUALS, ENTERING PARTS OF THE RESTRICTED AREA, WERE EXPOSED TO INTENSITIES OF BETA RADIATION UP TO 1500 MRAD/HR AND A SURVEY WAS NOT PERFORMED TO DETERMINE IF APPROPRIATE PERSONNEL MONITORING DEVICES WERE REQUIRED OR PROVIDED PURSUANT TO 10 CFR 20.202.
(8003 INFRACTION)

TECHNICAL SPECIFICATION 6.11 REQUIRES THAT PROCEDURES FOR PERSONNEL RADIATION PROTECTION SHALL BE PREPARED CONSISTENT WITH THE REQUIREMENTS OF PART 20 AND SHALL BE ADHERED TO FOR ALL OPERATIONS INVOLVING PERSONNEL RADIATION EXPOSURE. 1. PROCEDURE 902.7, REVISION 1, REQUIRES IN SECTION 5.4 THAT THE ALARM SET POINT OF THE RADIATION MONITORS SHALL HAVE ALARM SET POINTS SET BETWEEN TWO AND FIVE TIMES THE VALUE OF THE BACKGROUND IN THE AREA OF THE DETECTOR. DURING FUEL MOVEMENT ON JANUARY 24, 1980, AT 9:00 P.M. ONE OF THE TWO RADIATION MONITORS, USED FOR EVACUATION PURPOSES, WAS FOUND TO HAVE AN ALARM SET POINT TWELVE (12) TIMES THE VALUE OF THE BACKGROUND IN THE AREA OF THE DETECTOR. 2. PROCEDURE 915.7, REVISION 0, STATES IN SECTION 5.3, "THE TOLD AND SELF READING DOSIMETERS SHALL NORMALLY BE WORN IN THE FRONT OF THE BODY BETWEEN THE HEAD AND WAIST. ON JANUARY 21, 1980 ONE INDIVIDUAL WAS OBSERVED WEARING HIS TLD AND SELF READING DOSIMETERS SUSPENDED ABOUT THREE INCHES BELOW AND BEHIND HIS BELT AGAINST THE LEFT REAR POCKET. 3. PROCEDURE 9.11.1, REVISION 0, STATES IN PROCEDURE SECTION 5.1, THAT "ALL RADIOACTIVE WASTE MATERIAL SHALL BE STORED IN SUCH A MANNER SO AS TO REDUCE TO A MINIMUM: ...5.1.2 THE RADIATION EXPOSURE DOSE RATES FOR WORKING PERSONNEL." ON JANUARY 24, 1980, RADIOACTIVE WASTE WAS FOUND AT THE PERSONNEL ENTRANCE TO THE TORUS AND ON THE TORUS CATWALK EMANATING CONTACT DOSERATES OF 43 MILLIREM/HR. AND 3000 MILLIREM/HR. RESPECTIVELY. PERSONNEL, IN GOING TO AND FROM THEIR WORK LOCATIONS, REPEATEDLY PASSED BY THE DRUMS OF WASTE AND WERE AT TIMES OBSERVED TO BE LEANING AGAINST THE DRUMS AT THE TORUS ENTRANCE.
(8003 INFRACTION)

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-016/01T	05/12/80	05/23/80	Defective main generator load reject sensor pressure switch
80-017/01T	05/16/80	05/30/80	Rod block bypass jumpers (2) were left in place. Administrative control checks prevented more than 1 rod being moved during refueling.
80-020/01T	06/05/80	06/18/80	Identification of degraded fire barriers and failure to establish required fire watch.

CONTRARY TO 10 CFR 50.55(A)(G)(4), AS OF 3/20/80, PUMP & VALVE OPERABILITY TESTS REQUIRED BY ASME SECTION XI WERE NOT COMPLETED NOR HAVE PROCEDURES BEEN APPROVED TO IMPLEMENT THE REQUIRED TESTS.
(8010 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

- SYSTEMS AND COMPONENT PROBLEMS:
- Additional crack indications (2) have been identified by the licensee in Core Spray System II. Evaluation by the licensee and the NRC is in progress.

PLANT STATUS:

- The plant is shutdown for refueling with return to operation scheduled for the week of July 7, 1980.
- LAST IE SITE INSPECTION DATE: 6/30/80
- INSPECTION REPORT NO: 50-219/80-24

SUMMARY

INSPECTION SUMMARY

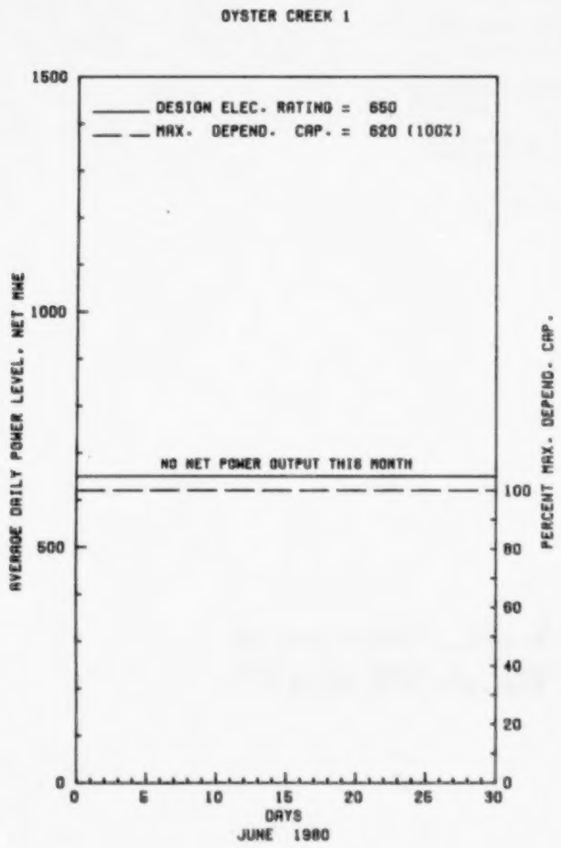
- 50-219/79-25 - June 25: Special enforcement meeting convened by Region I management to discuss NRC concerns relating to NRC Radiation Protection Inspection No. 50-219/79-07. The meeting was held to discuss licensee immediate corrective actions and long term plans for improving the licensee's Radiation Protection Program.
- 50-219/80-06 - Feb 4-8: Routine, unannounced inspection by one regional based inspector (40 hours) of activities associated with the inservice inspection program. No items of noncompliance were identified.
- 50-219/80-16 - May 5-9: Routine, unannounced inspection by one regional based inspector (37 hours) of licensee action on previous inspection findings; administrative controls for facility procedures; conformance to Technical Specifications; temporary and permanent changes in conformance to Technical Specifications and licensee procedures; changes in procedures to 10 CFR 50.59(a) and (b) requirements; checklists and related forms for currency to latest changes; and, control room tour. No items of noncompliance were identified.

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 8-4-80

REVIEWED BY NRR DL: W.A. Paulson **DATE:** 8-5-80

OYSTER CREEK

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-219
2. Utility Contact: C. M. McClain (201) 455-8748
3. Licensed Thermal Power (MWt): 1930
4. Nameplate Rating (Gross MWe): 687.5 x 0.8 = 550
5. Design Electrical Rating (Net MWe): 650
6. Maximum Dependable Capacity (Gross MWe): 650
7. Maximum Dependable Capacity (Net MWe): 620
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

ITEM 21 CALCULATED BY USING A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	4,367.0	92,231.0
12. Hours Reactor Was Critical	.0	101.5	68,740.8
13. Reactor Reserve Shutdown Hours	.0	.0	468.2
14. Hours Generator On-Line	.0	98.4	67,403.9
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	157,611	114,036,769
17. Gross Electrical Energy (MWH)	0	53,670	38,935,985
18. Net Electrical Energy (MWH)	0	48,612	37,529,625
19. Unit Service Factor	.0	2.3	73.1
20. Unit Availability Factor	.0	2.3	73.1
21. Unit Capacity Factor (MDC Net)	.0	1.8	67.2
22. Unit Capacity Factor (DER Net)	.0	1.7	62.6
23. Unit Forced Outage Rate	.0	.0	6.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): OCT. 11, 1980, 5 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: 07/12/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
1	01/05/80	S	720.0	C	4		RC	FUELXX	REFUELING/MAINTENANCE OUTAGE.

SUMMARY

THE PLANT WAS SHUTDOWN ALL MONTH FOR REFUELING AND MAINTENANCE.

LEGEND

[1] Type:
F–Forced
S–Scheduled

[2] Reason:
A–Equipment Failure
B–Maintenance or Test
C–Refueling
D–Regulatory Restriction
E–Operator Training & License Examination
F–Administrative
G–Operational Error
H–Other

[3] Method:
1–Manual
2–Manual Scram
3–Automatic Scram
4–Continuation
5–Load Reduction
9–Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

PALISADES

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Michigan/Van Buren 5 miles S of South Haven
2.	Electric Reliability Council	East Central Area Reliability Coordination Agreement
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2530
5.	Design Electrical Rating (MWE Net)	805
6.	Date of Initial Criticality	May 24, 1971
7.	Date of Commercial Operation	December 31, 1971
8.	Condenser Cooling Method	Forced Draft Cooling Towers
9.	Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10.	Licensee	Consumers Power Company
11.	Corporate Address	212 W. Michigan Avenue Jackson, Michigan 49201
12.	Corporate Contact	R. B. DeWitt
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Combustion Engineering
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	D.C. Boyd
19.	IE Resident Inspector	B. Jorgenson (616) 764-8971
20.	Licensing Project Manager	T. Wambach
21.	Docket Number	50-255
22.	License Number/Date of Issue	DPR-20, 3/24/71
23.	Public Document Room	Kalamazoo Public Library 315 South Rose Street Reference Department Kalamazoo, Michigan 49006

INSPECTION STATUS

ENFORCEMENT STATUS

A SECONDARY PHYSICAL BARRIER DID NOT PROVIDE ADEQUATE PROTECTION.
(8?04 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

- Operating routinely.

LAST IE SITE INSPECTION DATE: April 23, 1980

INSPECTION REPORT NO: 80-07

SUMMARY

INSPECTION SUMMARY

- No inspections conducted.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

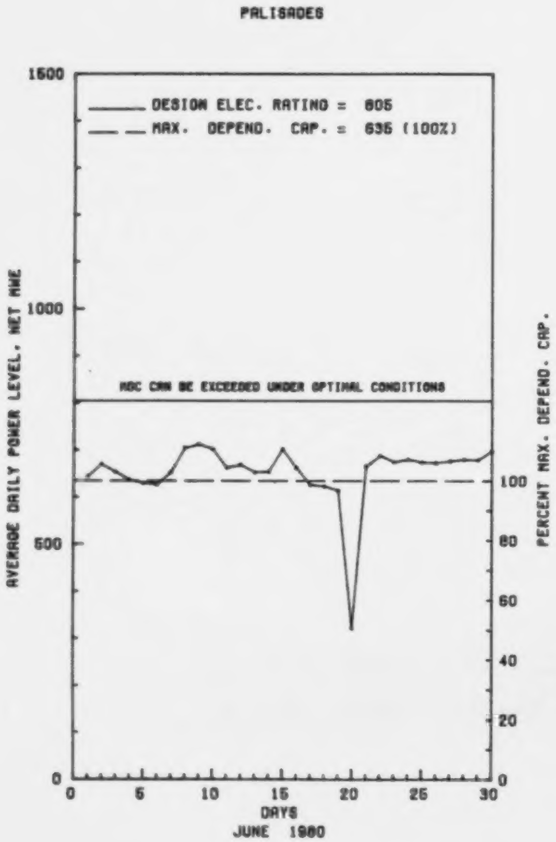
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-13/01T-0	05/21/80	06/06/80	HSPI flow transmitter inoperable.
80-14/03L-0	05/12/80	06/13/80	During routine testing of hydraulic snubbers, the oil level in snubber no. 5 was found to be less than zero, and snubber was declared inoperable.
80-15/03L-0	05/12/80	06/13/80	During routine testing of hydraulic snubbers, the oil level in snubber no. 9 was found to be less than zero, and snubber was declared inoperable.
80-16/03L-0	05/15/80	06/13/80	During routine testing of hydraulic snubbers, the oil level in snubber no. 7 was found to be less than zero, and the snubber was declared inoperable.
80-17/03L-0	05/18/80	06/13/80	Review of PS testing procedures revealed the setpoint of PS-1801 through 1804 to be lower than permitted by TS 3.16.
80-18/03L-0	05/22/80	06/19/80	During routine testing of hydraulic snubbers, the oil level in snubber no. 2 was found to be less than zero, and the snubber was declared inoperable.

REVIEWED BY OMPA LOEB: M R Beebe **DATE:** 08/04/80

REVIEWED BY NRR DL: T. V. Wambach **DATE:** 08/06/80

PALISADES

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-255
2. Utility Contact: P. J. Bodtke (616) 764-8913
3. Licensed Thermal Power (MWt): 2530
4. Nameplate Rating (Gross MWe): 955 x 0.85 = 812
5. Design Electrical Rating (Net MWe): 805
6. Maximum Dependable Capacity (Gross MWe): 675
7. Maximum Dependable Capacity (Net MWe): 635
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	4,367.0	74,798.0
12. Hours Reactor Was Critical	720.0	900.6	40,465.4
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	720.0	822.6	38,249.1
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,600,296	1,761,384	73,272,864
17. Gross Electrical Energy (MWH)	498,740	549,470	22,581,700
18. Net Electrical Energy (MWH)	470,439	516,627	21,187,402
19. Unit Service Factor	100.0	18.8	51.1
20. Unit Availability Factor	100.0	18.8	51.1
21. Unit Capacity Factor (MDC Net)	102.9	18.6	44.6
22. Unit Capacity Factor (DER Net)	81.2	14.7	35.2
23. Unit Forced Outage Rate	.0	.0	35.7

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
15	06/20/80	F	30.0	A	5		WE	HTEXCH	REDUCED POWER TO REPAIR CONDENSER LEAK.

SUMMARY

THE UNIT OPERATED NORMALLY WITH A BRIEF POWER REDUCTION ON 06/20/80 DUE TO REPAIRS TO CONDENSER LEAK.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

PEACH BOTTOM 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Pennsylvania/York 19 miles S of Lancaster
2.	Electric Reliability Council	Mid-Atlantic Area Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	3293
5.	Design Electrical Rating (MWE Net)	1065
6.	Date of Initial Criticality	September 16, 1973
7.	Date of Commercial Operati	July 5, 1974
8.	Condenser Cooling Method	Once Through with Cooling Towers
9.	Condenser Cooling Water Source	Conowingo Pond/Susquehanna River

Utility & Contractor Information

10.	Licensee	Philadelphia Electric Company
11.	Corporate Address	2301 Market Street Philadelphia, Pennsylvania 19101
12.	Corporate Contact	S. L. Daltroff, Vice President Electric Production
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	GE
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	IE Section Chief	E. McCabe
19.	IE Resident Inspector	C. Cowgill 717-456-7614
20.	Licensing Project Manager	D. Verrelli
21.	Docket Number	50-277
22.	License Number/Date of Issue	DPR-44, 8/8/73
23.	Public Document Room	Government Publications Section State Library of Pennsylvania Education Building Commonwealth and Walnut Street Harrisburg, Pennsylvania 17126

INSPECTION STATUS

ENFORCEMENT STATUS

TECHNICAL SPECIFICATION 6.8.1 STATES, IN PART, "WRITTEN PROCEDURES AND ADMINISTRATIVE POLICIES SHALL BE ESTABLISHED, IMPLEMENTED AND MAINTAINED...." CONTRARY TO THE ABOVE, THE FOLLOWING EXAMPLE OF FAILURE TO FOLLOW PROCEDURES WERE IDENTIFIED. (1) STATION PROCEDURE S.3.2.A.1, REVISION 4, DATED MAY 15, 1978, PROVIDES A CHECK-OFF LIST, S.3.2.A.1.A.2, FOR POSITIONING THE RESIDUAL HEAT REMOVAL SYSTEM MANUAL VALVES FOR OPERATION. ON APRIL 12, 1979, THE UNIT 2 RHR PRESSURIZING TO HEAD SPRAY HEADER INNER VALVE WAS FOUND TO BE INCORRECTLY POSITIONED AND NOT IN ACCORDANCE WITH CHECK OFF LIST S.3.2.A.1.A.2. (2) STATION PROCEDURE S.3.2.A.1, REVISION 4, DATED MAY 15, 1978, PROVIDES A CHECK-OFF LIST, S.3.2.A.1.A.2, FOR POSITIONING THE RESIDUAL HEAT REMOVAL SYSTEM MANUAL VALVES FOR OPERATION. ON APRIL 12, 1979, THE UNIT 2 RHR PRESSURIZING TO SHUTDOWN COOLING SUCTION HEADER OUTER VALVE WAS FOUND TO BE INCORRECTLY POSITIONED AND NOT IN ACCORDANCE WITH CHECK-OFF LIST S.3.2.A.1.A.2. (3) STATION PROCEDURE, S.8.4.A, REVISION 3, DATED JULY 13, 1977 PROVIDES A CHECK-OFF LIST, S.8.4.A, FOR MAKING THE APPROPRIATE MECHANICAL AND ELECTRICAL CHECKS TO SET UP THE DIESEL GENERATORS LOCALLY FOR AUTOMATIC OPERATION. ON APRIL 12, 1979, TWO OF SEVEN VALVES REQUIRED TO BE LOCKED IN ACCORDANCE WITH S.8.4.A WERE FOUND TO BE NOT LOCKED. (RECURRENT)
(7911 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

MANAGERIAL ITEMS:

- Members of licensee management met with NRC Region I staff on June 19, 1980 to discuss licensee performance. This meeting was conducted as part of the systematic/assessment of licensee performance (SALP) program.
- The NRC conducted a health physics team inspection at the facility June 16-27.

PLANT STATUS:

- The unit remained shutdown for refueling and torus modifications throughout the month.

- LAST IE SITE INSPECTION DATE: 6/30/80
- INSPECTION REPORT NO: 50-277/80-20

SUMMARY

INSPECTION SUMMARY

- 50-277/80-17 - June 4-5: Routine, announced inspection by one regional based inspector (31 hours) of the components identified in the Main Steam and Feedwater System of licensee's response to NRC Bulletin 79-01B dated April 15, 1980. No items of noncompliance were identified.

REPORTS RECEIVED FROM LICENSEE

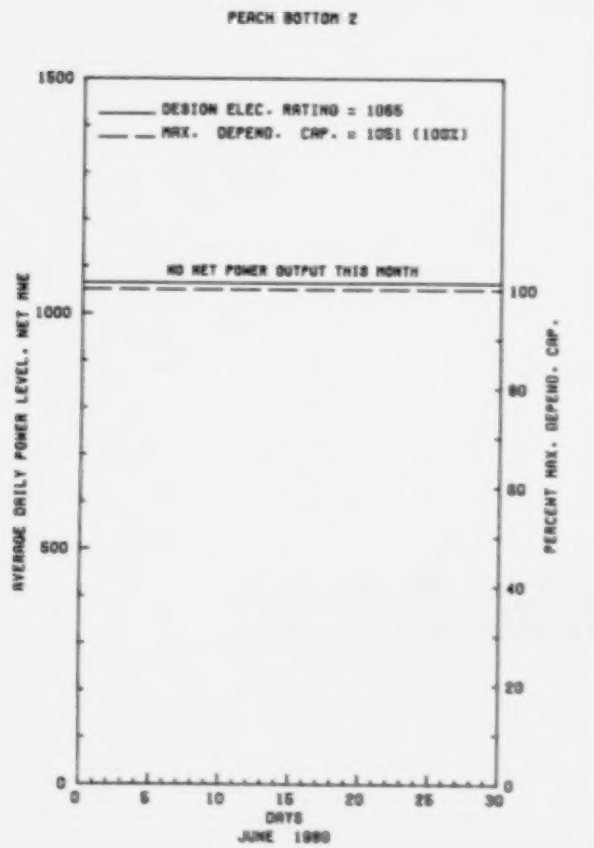
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
NONE				

REVIEWED BY OMPA LOEB: John J. Rizzo DATE: 8-4-80

REVIEWED BY NRR DL: DATE: 8-5-80

PEACH BOTTOM 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-277
2. Utility Contact: W. M. Alden (215) 841-5022
3. Licensed Thermal Power (MWt): 3293
4. Nameplate Rating (Gross MWe): 1280 x 0.9 = 1152
5. Design Electrical Rating (Net MWe): 1065
6. Maximum Dependable Capacity (Gross MWe): 1098
7. Maximum Dependable Capacity (Net MWe): 1051
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	4,367.0	52,511.0
12. Hours Reactor Was Critical	0	1,441.0	39,196.0
13. Reactor Reserve Shutdown Hours	0	0	0
14. Hours Generator On-Line	0	1,390.2	38,373.0
15. Unit Reserve Shutdown Hours	0	0	0
16. Gross Thermal Energy (MWH)	0	3,831,770	110,597,616
17. Gross Electrical Energy (MWH)	0	1,255,980	36,316,210
18. Net Electrical Energy (MWH)	-7,033	1,181,770	34,797,563
19. Unit Service Factor	0	31.8	73.1
20. Unit Availability Factor	0	31.8	73.1
21. Unit Capacity Factor (MDC Net)	0	25.7	63.1
22. Unit Capacity Factor (DER Net)	0	25.4	62.2
23. Unit Forced Outage Rate	0	.9	6.1

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 07/22/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
9	03/21/80	S	720.0	C	4		RC	FUELXX	CONTINUATION OF REFUEL OUTAGE.

SUMMARY

THE PLANT WAS SHUTDOWN ALL MONTH FOR REFUELING.

LEGEND

[1] Type:
F–Forced
S–Scheduled

[2] Reason:
A–Equipment Failure
B–Maintenance or Test
C–Refueling
D–Regulatory Restriction
E–Operator Training & License Examination
F–Administrative
G–Operational Error
H–Other

[3] Method:
1–Manual
2–Manual Scram
3–Automatic Scram
4–Continuations
5–Load Reduction
9–Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

PEACH BOTTOM 3

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Pennsylvania/York 19 miles S of Lancaster
2.	Electric Reliability Council	Mid-Atlantic Area Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	3293
5.	Design Electrical Rating (MWE Net)	1065
6.	Date of Initial Criticality	August 7, 1974
7.	Date of Commercial Operation	December 23, 1974
8.	Condenser Cooling Method	Once Through with Cooling Towers
9.	Condenser Cooling Water Source	Conowingo Pond/Susquehanna River

Utility & Contractor Information

10.	Licensee	Philadelphia Electric Company
11.	Corporate Address	2301 Market Street Philadelphia, Pennsylvania 19101
12.	Corporate Contact	S. L. Daltroff, Vice President Electric Production
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	GE
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	IE Section Chief	E. McCabe
19.	IE Resident Inspector	C. Cowgill (717-456-7614)
20.	Licensing Project Manager	D. Verrelli
21.	Docket Number	50-278
22.	License Number/Date of Issue	DPR-56, 7/2/74
23.	Public Document Room	Government Publications Section State Library of Pennsylvania Education Building Commonwealth and Walnut Street Harrisburg, Pennsylvania 17126

INSPECTION STATUS

ENFORCEMENT STATUS

TECHNICAL SPECIFICATION 6.8.3 STATES, IN PART, "TEMPORARY CHANGES TO PROCEDURES OF 6.8.1 ABOVE MAY BE MADE, PROVIDED: B. THE CHANGE IS APPROVED BY TWO MEMBERS OF THE PLANT MANAGEMENT, AT LEAST ONE OF WHOM HOLDS A SENIOR REACTOR OPERATOR'S LICENSE ON THE AFFECTED UNIT. C. THE CHANGE IS DOCUMENTED, REVIEWED BY THE PORC AND APPROVED BY THE STATION SUPERINTENDENT WITHIN 14 DAYS OF IMPLEMENTATION." CONTRARY TO THE ABOVE, ON AUGUST 11, 1978, A TEMPORARY PROCEDURE CHANGE WAS MADE TO SURVEILLANCE TEST, ST 1.3, REVISION 3, DATED MAY 9, 1978, PCIS LOGIC SYSTEM FUNCTIONAL TEST-UNIT 3, WITHOUT COMPLETE APPROVALS. FURTHER, SUBSEQUENT PORC REVIEW HAD NOT BEEN ACCOMPLISHED AS OF APRIL 13, 1979. (7912 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

MANAGERIAL ITEMS:

- Members of licensee management met with NRC Region I staff on June 19, 1980 to discuss licensee performance. This meeting was conducted as part of the systematic assessment of licensee performance (SALP) program.
- The NRC conducted a health physics team inspection June 16-27.

PLANT STATUS:

- The unit was started up June 3 following recirculation pump seal replacement and subsequently operated at a nominal 100 percent power. Representative combined Unit 2 and Unit 3 stack gas release rate was 180 microcuries per second.
- LAST IE SITE INSPECTION DATE: 6/30/80
- INSPECTION REPORT NO: 50-278/80-14

SUMMARY

INSPECTION SUMMARY

No reports issued.

REPORTS RECEIVED FROM LICENSEE

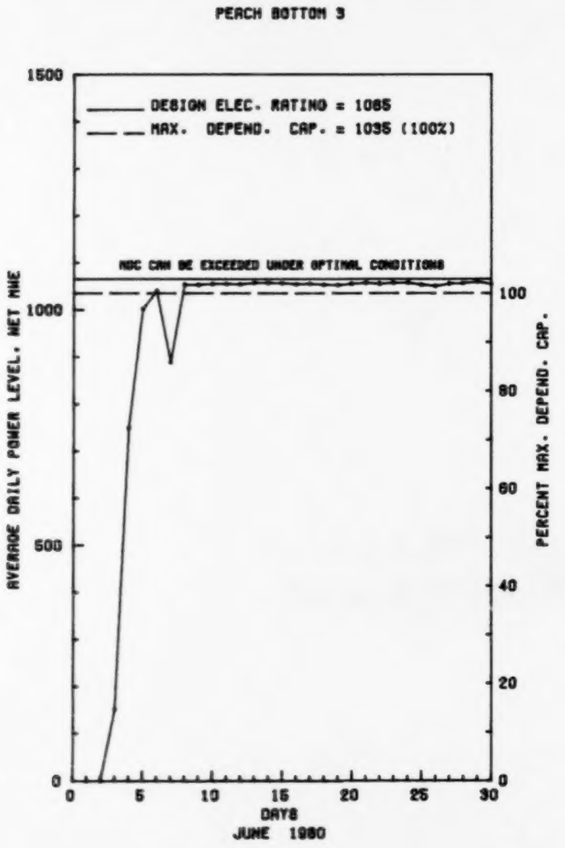
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-010/03L	04/21/80	05/21/80	LPCI MOV did not open when tested but opened on next try and later extra tests. During shutdown, disc to guide clearance lack was fixed and MOV tested ok after leak rate testing.

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 8-4-80

REVIEWED BY NRR DL: **DATE:** 8-5-80

PEACH BOTTOM 3

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-278
2. Utility Contact: W. M. Alden (215) 841-5022
3. Licensed Thermal Power (MWt): 3293
4. Nameplate Rating (Gross MWe): 1280 x 0.9 = 1152
5. Design Electrical Rating (Net MWe): 1065
6. Maximum Dependable Capacity (Gross MWe): 1098
7. Maximum Dependable Capacity (Net MWe): 1035
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	4,367.0	48,487.0
12. Hours Reactor Was Critical	671.0	3,358.0	38,284.0
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	659.8	3,216.4	37,148.2
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	2,118,218	10,210,301	104,429,455
17. Gross Electrical Energy (MWH)	699,870	3,420,400	34,006,260
18. Net Electrical Energy (MWH)	673,842	3,296,504	32,641,304
19. Unit Service Factor	91.6	73.7	76.7
20. Unit Availability Factor	91.6	73.7	76.7
21. Unit Capacity Factor (MDC Net)	90.4	72.9	65.2
22. Unit Capacity Factor (DER Net)	87.9	70.9	63.3
23. Unit Forced Outage Rate	8.4	18.3	8.0

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
12	06/01/80	F	60.2	A	1		CB	PUMPXX	CONT. OF 38 REACTOR RECIRC. PUMP SEAL REPLACEMENT.

SUMMARY

THE REACTOR REACHED FULL POWER ON 06/05/80 FOLLOWING PUMP SEAL REPAIR AND OPERATED AT THAT LEVEL THROUGHOUT JUNE EXCEPT FOR A REDUCTION ON 06/16/80 FOR CONTROL ROD ADJUSTMENT.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

PILGRIM

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Massachusetts/Plymouth 4 miles SE of Plymouth
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	BWR
4.	License Power Level (MWT)	1998
5.	Design Electrical Rating (MWE Net)	655
6.	Date of Initial Criticality	June 16, 1972
7.	Date of Commercial Operation	December 1972
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Cape Cod Bay

Utility & Contractor Information

10.	Licensee	Boston Edison Company
11.	Corporate Address	800 Boylston Street Boston, Massachusetts 02199
12.	Corporate Contact	A. Corey, Vice President
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	GE
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	IE Section Chie.	T. MARTIN
19.	IE Resident Inspector	J. JOHNSON
20.	Licensing Project Manager	M. H. Williams
21.	Docket Number	50-293
22.	License Number/Date of Issue	DPR-35, 6/8/72
23.	Public Document Room	Plymouth Public Library North Street Plymouth, Massachusetts 02360

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO LICENSEE TRAINING MANUAL IMPLEMENTATION OF 10 CFR 55, APPENDIX A, PARA. 5, RECORDS, THE 1979 ANNUAL REVIEW BY THE STATION MANAGER WAS NOT RECORDED & THE 1979 ANNUAL EXAMINATION RESULTS WERE NOT RECORDED ON THE PILGRIM NUCLEAR POWER STATION LICENSED OPERATOR REQUALIFICATION SUMMARY RECORDS FOR THE 10 RECORDS REVIEWED.
(8014 DEFICIENCY)

TS 6.8.D STATES THAT "WRITTEN PROCEDURES TO IMPLEMENT THE FIRE PROTECTION PROGRAM SHALL BE ESTABLISHED, IMPLEMENTED & MAINTAINED." LICENSEE'S PROCEDURES #8A.6-1, FIRE EXTINGUISHER CHECKS, REV. 5, 8/9/78 STATES IN PART, "PERFORM A FIRE EXTINGUISHER CHECK ON EXTINGUISHERS ONCE PER MONTH...THE INSPECTION PROCEDURE SHOULD DETERMINE THAT: 1. THE EXTINGUISHER IS IN ITS DESIGNATED PLACE. 2. ACCESS OR VISIBILITY OF THE EXTINGUISHER IS NOT OBSTRUCTED... 6. MAINTENANCE RECOD TAG IS UP TO DATE...." CONTRARY TO THE ABOVE, AS OF 4/25/80, FIRE EXTINGUISHERS HAVE BEEN EITHER REMOVED FROM THEIR DESIGNATED PLACES OR THE EXTINGUISHERS WERE FOUND TO BE OBSTRUCTED BY TEMPORARY EQUIPMENT, THUS LEAVING VITAL PLANT AREAS W/O ADEQUATE FIRST AID FIRE PROTECTION. MAINTENANCE RECORD TAGS ON MOST EXTINGUISHERS WERE NOT UP TO DATE.
(8016 INFRACTION)

FAILURE TO USE ESTABLISHED CONTROLS FOR FOREIGN MATERIAL AS SPECIFIED IN MAINTENANCE PROCEDURE.
(8018 DEFICIENCY)

CONTRARY TO 10 CFR 50, APPENDIX J, SECTION III.A.1.(A), DURING THE TYPE A TEST ONE REPAIR AND ONE ADJUSTMENT TO THE CONTAINMENT ISOLATION BOUNDARY WERE MADE WITHOUT MEASURING (QUANTIFYING) THE "AS FOUND" LEAKAGE. THEY ARE: (1) ATTEMPTED REPAIR (TIGHTENING) OF THE LEAKING 8 INCH PIPE FLANGE ON THE RCIC TURBINE EXHAUST LINE ON MAY 5, 1980. (2) MADE ADJUSTMENT BY CLOSURE OF VALVE AO5033A IN THE CONTAINMENT ATMOSPHERIC CONTROL SYSTEM ON MAY 6, 1980.
(8020 INFRACTION)

CONTRARY TO 10 CFR 50, APPENDIX J, SECTION IV, A AND STATION PROCEDURE 8.7.1.3, AN 8 INCH PIPE FLANGE ON THE RCIC TURBINE EXHAUST LINE, WHICH IS PART OF THE PRIMARY REACTOR CONTAINMENT BOUNDARY, WAS DISASSEMBLED FOR MAINTENANCE ON MAY 2, 1980 AND WAS NOT RETESTED TO VERIFY THAT THE COMBINED LEAKAGE RATE FOR PENETRATIONS AND VALVES WAS WITHIN THE ALLOWABLE LIMITS.
(8020 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS:

- The plant is presently operating at full power with representative stack and offgas release rates of 6 and 17,000 microcuries/sec., respectively.

LAST IE SITE INSPECTION DATE: 5/31/80

INSPECTION REPORT NO: 50-293/80-23

SUMMARY

INSPECTION SUMMARY

- 50-293/80-16 - Apr 21-25: Announced inspection by one regional based inspector (32 hours) of the plant Fire Protection/Prevention Program, including: implementation of administrative controls; fire brigade training; observation of ignition source and combustible materials control; review and observation of plant modifications; and observation of critical plant areas One item of noncompliance was identified: Failure to follow plant fire extinguishers check procedure.
- 50-293/80-19 - Apr 28 - May 2: Investigation by one regional based investigator (38 hours) of allegations contained in news paper articles dated May 20, april 3, and April 8, 1980. None of the allegations investigated were found to be substantiated. No items of noncompliance were identified.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-017/03L	05/05/80	06/02/80	Three unsuccessful attempts to perform 24 hour test of diesel generator pursuant to IEB 79-23.

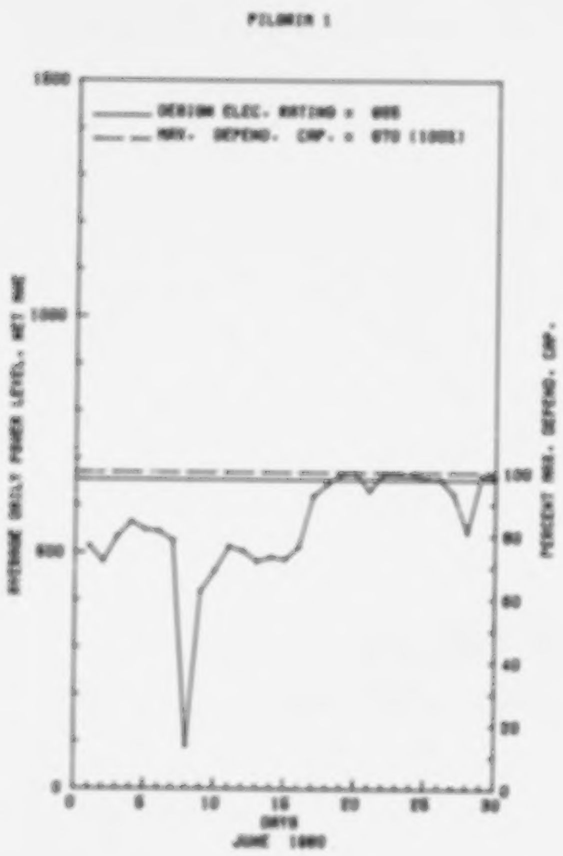
- 50-293/80-20 - May 4-8: Routine unannounced inspection by one regional based inspector (42 hours) of Containment Leak Rate Testing and licensee action on previous inspection findings. Two items of noncompliance: Failure to conduct CILRT in accordance with 10 CFR 50, Appendix J; and, failure to perform local leak rate test following maintenance.
- 50-293/80-24 - Apr 9-10: Routine, announced inspection by one regional based inspector (12 hours) of the installed Main Steam ADS System Relief Valve Solenoid Valves including cable, raceways, junction boxes and penetrations as described in the IEB 79-01B licensee submittal dated April 1, 1980. No items of noncompliance were identified.

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 8-4-80

REVIEWED BY NRR DL: John N. Hannon **DATE:** 8-6-80

PILGRIM

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-293
2. Utility Contact: G. G. Whitney (617) 746-7900
3. Licensed Thermal Power (MWt): 1998
4. Nameplate Rating (Gross MWe): 788 x 0.87 = 678
5. Design Electrical Rating (Net MWe): 655
6. Maximum Dependable Capacity (Gross MWe): 690
7. Maximum Dependable Capacity (Net MWe): 670
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons For Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	4,367.0	66,263.1
12. Hours Reactor Was Critical	711.0	1,159.2	46,129.9
13. Reactor Reserve Shutdown Hours	0	0	0
14. Hours Generator On-Line	702.3	1,044.6	44,619.1
15. Unit Reserve Shutdown Hours	0	0	0
16. Gross Thermal Energy (MWH)	1,321,128	1,735,528	76,821,440
17. Gross Electrical Energy (MWH)	436,870	556,740	24,751,574
18. Net Electrical Energy (MWH)	421,031	541,775	23,769,898
19. Unit Service Factor	97.5	24.4	67.3
20. Unit Availability Factor	97.5	24.4	67.3
21. Unit Capacity Factor (MDC Net)	83.1	19.3	53.5
22. Unit Capacity Factor (DER Net)	85.2	19.7	54.8
23. Unit Forced Outage Rate	2.5	3.9	10.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
3	05/26/80	F	.0	H	5		HG	DEMINX	HIGH D/P ACROSS CONDENSATE DEMINERALIZERS DUE TO RUST BUILD-UP IN LINES DURING OUTAGE.
4	06/08/80	F	17.7	H	3		HC	HTEXCH	REACTOR SCRAMMED AT 0451 ON 06/08/80 DUE TO LOW VACUUM DURING CONDENSER BACKWASH.

SUMMARY

THE UNIT BEGAN THE REPORTING PERIOD AT REDUCED POWER DUE TO HIGH D/P ACROSS CONDENSATE DEMINERALIZERS. ON 06/08/80, A SCRAM OCCURRED DUE TO LOW VACUUM DURING CONDENSER BACKWASH. ON 06/17/80 THE UNIT WAS BACK TO FULL POWER AND RAN NORMALLY FOR THE REST OF THE MONTH.

LEGEND

[1] Type:
F–Forced
S–Scheduled

[2] Reason:
A–Equipment Failure
B–Maintenance or Test
C–Refueling
D–Regulatory Restriction
E–Operator Training & License Examination
F–Administrative
G–Operational Error
H–Other

[3] Method:
1–Manual
2–Manual Scram
3–Automatic Scram
4–Continuations
5–Load Reduction
9–Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

POINT BEACH 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Wisconsin/Manitowoc 15 miles N of Manitowoc
2.	Electric Reliability Council	Mid-America Interpool Network
3.	Type of Reactor	PWR
4.	Power Level (MWT)	1518
5.	Design Electrical Rating (MWE Net)	497
6.	Date of Initial Criticality	November 2, 1970
7.	Date of Commercial Operation	December 21, 1970
8.	Condenser Cooling Method	Straight Through
9.	Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10.	Licensee	Wisconsin Electric Power Co.
11.	Corporate Address	231 West Michigan Milwaukee, Wisconsin
12.	Corporate Contact	S. Burstain
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	R. F. Warnick
19.	IE Resident Inspector	W. Guldemond
20.	Licensing Project Manager	C. M. Trammell
21.	Docket Number	50-266
22.	License Number/Date of Issue	DPR-24 issued 10/05/70
23.	Public Document Room	Joseph Mann Public Library Two Rivers, Wisconsin Manitowoc Public Library Manitowoc, Wisconsin (Temporary LPDR for spent fuel pool capacity increase amendment only

INSPECTION STATUS

ENFORCEMENT STATUS

SOME VITAL AREA PORTALS WERE NOT SECURED AS REQUIRED BY THE APPROVED SECURITY PLAN.
(8006 INFRACTION)

A CHANGE IN ALARM STATUS WAS NOT PROPERLY MONITORED.
(8006 INFRACTION)

THE ALARM CIRCUITRY FOR SOME VITAL AREA PORTALS WAS NOT TESTED AT THE REQUIRED FREQUENCY.
(8006 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

The unit operated routinely during the month.

+LAST IE SITE INSPECTION DATE: May 19-23, 1980

+ INSPECTION REPORT NO: 80-08

SUMMARY

INSPECTION SUMMARY

+ Inspection on May 19-23, (80-08): Material Control and Accountability including facility organization and operation; measurement and control; storage and internal controls; shipping and receiving; records and reports; inventory and inventory verification; and management of material control system. The inspection involved 60 inspector hours onsite by two NRC inspectors and was begun during the regular hours. The licensee was found to be in compliance with NRC requirements in the seven areas examined during the inspection.

REPORTS RECEIVED FROM LICENSEE

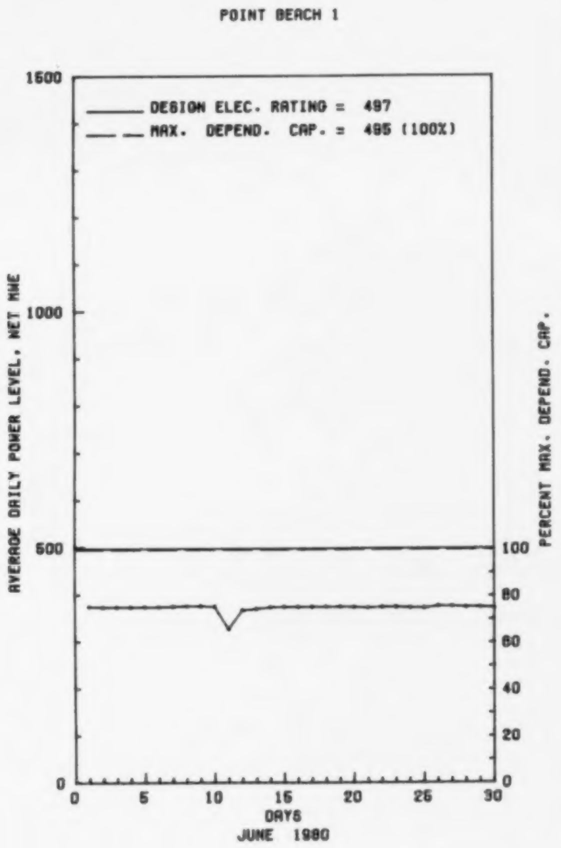
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-05/03L-0	04/17/80	06/02/80	Designated fire brigade below the minimum manning requirements.

REVIEWED BY OMPA LOEB: Earl C. Hill **DATE:** 8-4-80

REVIEWED BY NRR DL: PC Wagner **DATE:** 8/5/80

POINT BEACH 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-266
2. Utility Contact: C.W. FAY (414) 277-2811
3. Licensed Thermal Power (MWt): 1518
4. Nameplate Rating (Gross MWe): 582 x 0.9 = 524
5. Design Electrical Rating (Net MWe): 497
6. Maximum Dependable Capacity (Gross MWe): 519
7. Maximum Dependable Capacity (Net MWe): 495
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): 390
10. Reasons for Restrictions, If Any: REDUCTION IN CORE AVE. TEMP. IN ATTEMPT TO INHIBIT CORROSION OF SG TUBES.

ITEM 21 CALCULATED BY USING A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	4,367.0	84,599.0
12. Hours Reactor Was Critical	718.6	3,617.1	70,133.6
13. Reactor Reserve Shutdown Hours	.3	19.1	601.3
14. Hours Generator On-Line	717.8	3,473.3	67,867.3
15. Unit Reserve Shutdown Hours	0	151.5	753.9
16. Gross Thermal Energy (MWH)	870,132	4,185,056	94,937,851
17. Gross Electrical Energy (MWH)	282,500	1,364,180	31,997,930
18. Net Electrical Energy (MWH)	267,047	1,288,840	30,479,018
19. Unit Service Factor	99.7	79.5	80.2
20. Unit Availability Factor	99.7	83.0	81.1
21. Unit Capacity Factor (MDC Net)	74.9	59.6	74.8
22. Unit Capacity Factor (DER Net)	74.6	59.4	72.5
23. Unit Forced Outage Rate	.3	.1	3.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): UNIT SCHED TO SHUTDOWN 7/25/80 FOR S/G LEAK TESTING.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
2	06/11/80	F	2.2	G	3	80-007/01T-0	EC	GENERA	THE REMOVAL OF A VITAL BUS SUPPLY INVERTER FROM SERVICE WITH A COMMUNICATIONS MIXUP, AND SUBSEQUENT UPSET OF SECONDARY PLANT CONDITIONS CAUSING IMPROPER FEEDWATER CONTROL AND STEAM/FEEDWATER FLOW MISMATCH CAUSED UNIT TRIP. ADDITIONAL ADMINISTRATIVE CONTROLS WILL BE INITIATED TO HELP PREVENT FURTHER EVENTS.

SUMMARY

THE UNIT OPERATED NORMALLY WITH ONLY A BRIEF OUTAGE ON 06/11/80 DUE TO STEAM/FEEDWATER FLOW MISMATCH.

LEGEND

[1] Type:
- F–Forced
- S–Scheduled

[2] Reason:
- A–Equipment Failure
- B–Maintenance or Test
- C–Refueling
- D–Regulatory Restriction
- E–Operator Training & License Examination
- F–Administrative
- G–Operational Error
- H–Other

[3] Method:
- 1–Manual
- 2–Manual Scram
- 3–Automatic Scram
- 4–Continuations
- 5–Load Reduction
- 9–Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source

POINT BEACH 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Wisconsin/Manitowoc 15 miles N of Manitowoc
2.	Electric Reliability Council	Mid-America Interpool Network
3.	Type of Reactor	PWR
4.	Power Level (MWT)	1518
5.	Design Electrical Rating (MWE Net)	497
6.	Date of Initial Criticality	May 30, 1972
7.	Date of Commercial Operation	October 1, 1972
8.	Condenser Cooling Method	Straight Through
9.	Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10.	Licensee	Wisconsin Electric Power Co.
11.	Corporate Address	231 West Michigan Milwaukee, Wisconsin
12.	Corporate Contact	S. Burstein
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	R. F. Warnick
19.	IE Resident Inspector	W. Guldemond
20.	Licensing Project Manager	C. M. Trammell
21.	Docket Number	50-301
22.	License Number/Date of Issue	DPR-27 issued 11/18/71
23.	Public Document Room	Joseph Mann Public Library Two Rivers, Wisconsin Manitowoc Public Library Manitowoc, Wisconsin (Temporary LPDR for spent fuel pool capacity increase amendment only

INSPECTION STATUS

ENFORCEMENT STATUS

SOME VITAL AREA PORTALS WERE NOT SECURED AS REQUIRED BY THE APPROVED SECURITY PLAN.
(8005 INFRACTION)

A CHANGE IN ALARM STATUS WAS NOT PROPERLY MONITORED.
(8005 INFRACTION)

THE ALARM CIRCUITRY FOR SOME VITAL AREA PORTALS WAS NOT TESTED AT THE REQUIRED FREQUENCY.
(8005 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

- The unit operated routinely during the month.

*LAST IE SITE INSPECTION DATE: May 27-29, 1980

- INSPECTION REPORT NO: 80-09

SUMMARY

INSPECTION SUMMARY

- Inspection on May 19-23, (80-07): Material Control and Accountability including facility organization and operation; measurement and control; storage and internal controls; shipping and receiving; records and reports; inventory and inventory verification; and management of material control system. The inspection involved 60 inspector hours onsite by two NRC inspectors and was begun during the regular hours. The licensee was found to be in compliance with NRC requirements in the seven areas examined during the inspection.

- Inspection on May 27-29, (80-09): Routine, unannounced inspection of core power distribution limits; reactor shutdown margin determination; isothermal temperature coefficient of reactivity measurement; control rod worth measurements; target axial flux difference calculation; determination of reactivity anomalies; and rod drop time test. The inspection involved 22 inspector hours onsite by two NRC inspectors. No items of noncompliance or deviations were identified.

REPORTS RECEIVED FROM LICENSEE

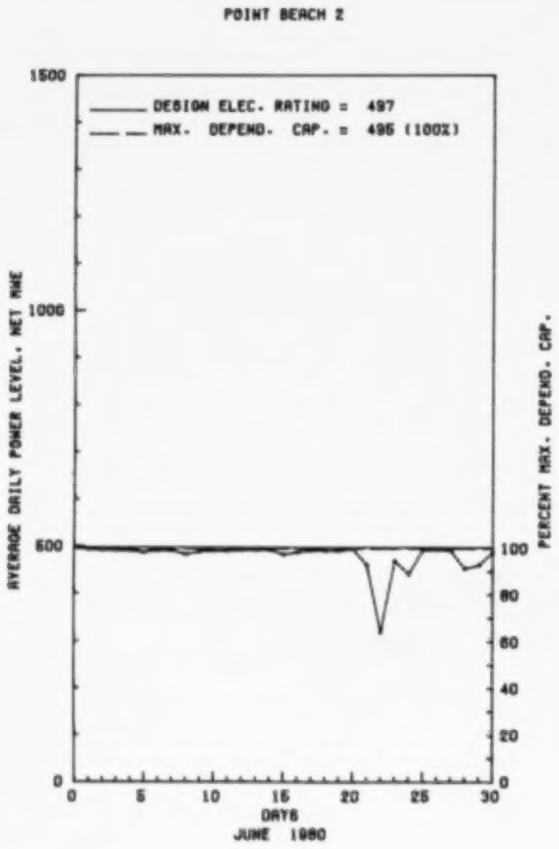
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
NONE				

REVIEWED BY OMPA LOEB: Earl Hall **DATE:** 8·4·80

REVIEWED BY NRR DL: PC Wagner **DATE:** 8/5/80

POINT BEACH 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-301
2. Utility Contact: C.W. FAY (414) 277-2811
3. Licensed Thermal Power (MWt): 1518
4. Nameplate Rating (Gross MWe): 582 x 0.9 = 524
5. Design Electrical Rating (Net MWe): 497
6. Maximum Dependable Capacity (Gross MWe): 519
7. Maximum Dependable Capacity (Net MWe): 495

ITEM 21 CALCULATED BY USING A WEIGHTED AVERAGE

8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	4,367.0	69,384.0
12. Hours Reactor Was Critical	720.0	3,266.9	62,163.8
13. Reactor Reserve Shutdown Hours	.0	19.2	185.3
14. Hours Generator On-Line	720.0	3,208.7	60,927.7
15. Unit Reserve Shutdown Hours	.0	13.5	119.9
16. Gross Thermal Energy (MWH)	1,045,235	4,434,807	82,257,363
17. Gross Electrical Energy (MWH)	361,060	1,547,850	27,909,190
18. Net Electrical Energy (MWH)	344,556	1,472,182	26,547,309
19. Unit Service Factor	100.0	73.5	87.8
20. Unit Availability Factor	100.0	73.8	88.0
*21. Unit Capacity Factor (MDC Net)	96.7	68.1	77.3
22. Unit Capacity Factor (DER Net)	96.3	67.8	77.0
23. Unit Forced Outage Rate	.0	9.9	2.0

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	NONE								

SUMMARY

THE UNIT OPERATED AT FULL POWER WITH A REDUCTION ON 06/22/80 FOR CONDENSER TUBE LEAK TESTING.

LEGEND

[1] Type:
F-Forced
S-Scheduled

[2] Reason:
A-Equipment Failure
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error
H-Other

[3] Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Continuations
5-Load Reduction
9-Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

PRAIRIE ISLAND 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Minnesota/Goodhue 28 miles SE of Minneapolis
2.	Electric Reliability Council	Mid-Continent Area Reliability Coordination Agreement
3.	Type of Reactor	PWR
4.	Power Level (MWT)	1650
5.	Design Electrical Rating (MWE Net)	530
6.	Date of Initial Criticality	December 1, 1973
7.	Date of Commercial Operation	December 16, 1973
8.	Condenser Cooling Method	Cooling Towers
9.	Condenser Cooling Water Source	Mississippi River

Utility & Contractor Information

10.	Licensee	Northern States Power Company
11.	Corporate Address	414 Nicollet Mall - 8th Floor Minneapolis, Minnesota 55401
12.	Corporate Contact	G. H. Neils (Oper.) L. O. Mayer (Lic.)
13.	Architect/Engineer	Pioneer Services & Eng. Co.
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Northern States Power Co.

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	W. S. Little
19.	IE Resident Inspector	C. D. Feierabend 612-388-8209
20.	Licensing Project Manager	R. Martin
21.	Docket Number	50-282
22.	License Number/Date of Issue	DPR-42 issued 8/9/73
23.	Public Document Room	Environmental Conservation Library Minneapolis Public Library 300 Nicollet Mall Minneapolis, Minnesota 55401

INSPECTION STATUS

ENFORCEMENT STATUS

YOUR RESPONSES TO IE BULLETIN NO. 78-05, DATED MAY 30, 1978 AND AUGUST 22, 1979, COMMIT THAT THE SPECIAL INSPECTIONS OF PLUNGER ARM CLEARANCE ON MOTOR STARTERS ON SAFETY RELATED SYSTEMS WOULD BE COMPLETED DURING THE NEXT REFUELING OUTAGE. BOTH UNITS HAVE SUBSEQUENTLY COMPLETED REFUELING OUTAGES. CONTRARY TO THE ABOVE, THE SPECIAL INSPECTIONS HAD NOT BEEN COMPLETED FOR THE MOTOR STARTERS ASSOCIATED WITH THE POWER OPERATED RELIEF VALVE ISOLATION VALVES FOR EITHER UNIT.
(8006 DEVIATION)

THE CORRECTED VISION OF SOME SECURITY FORCE PERSONNEL DID NOT MEET ESTABLISHED STANDARDS.
(8007 DEFICIENCY)

SOME VITAL AREA PHYSICAL BARRIERS WERE NOT PROPERLY MAINTAINED.
(8007 INFRACTION)

THE INTRUSION DETECTION SYSTEM DID NOT DETECT PENETRATION OF THE BOUNDARY AT SOME LOCATIONS.
(8007 DEFICIENCY)

AN UNATTENDED DESIGNATED VEHICLE IN THE PROTECTED AREA WAS NOT POSITIVELY CONTROLLED.
(8007 DEFICIENCY)

SOME ALARM DEVICES AND MONITORING SYSTEMS WERE NOT TAMPER-INDICATING.
(8007 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

+ Shutdown on July 1, 1980, due to a tube leak in steam generator 1-2. The unit will remain down to begin a refueling outage which was scheduled to begin on August 6, 1980.

+LAST IE SITE INSPECTION DATE: May 1-31, 1980

+ INSPECTION REPORT NO: 80-09

SUMMARY

INSPECTION SUMMARY

+ Meeting on February 26, (80-04): Discussed plant performance and enforcement history for the Prairie Island Plant, including the enforcement action resulting from inspection conducted in January 1980. Also discussed the implementation of the resident inspector program at the Prairie Island Plant.

+ Inspection on May 1-31, (80-09): Routine, resident inspection of plant operation, maintenance, surveillance, security, radiation protection, followup of licensee reported events, followup of IE Circulars and followup of licensee actions related to TMI lessons learned commitments. The inspection involved 87 inspector hours onsite by two NRC inspectors. No items of noncompliance or deviations were identified.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

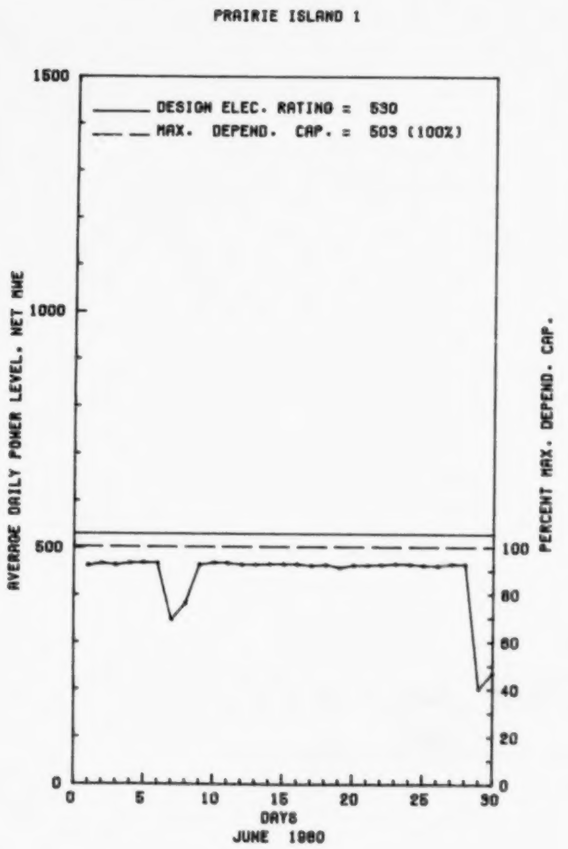
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-14/ 03L-0	04/10/80	05/09/80	During surveillance test, one overpower delta T setpoint was found to trip about 70% above the desired setpoint.
80-15/ 01T-0	04/29/80	05/13/80	A volumetric test of the containment maintenance airlock showed leakage of about 600 sccm vs. the TS limit of 409 sccm.
80-01		06/02/80	Maximum discharge temperature TS B-2.2.1. (Environmental Report)
80-02		06/09/80	pH of discharges TS b-2.4.3. (Environmental Report)

REVIEWED BY OMPA LOEB: Earl Hild **DATE:** 8·4·80

REVIEWED BY NRR DL: R Martin **DATE:** 8/5/80

PRAIRIE ISLAND 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-282
2. Utility Contact: A. A. Hunstad (612) 388-4767
3. Licensed Thermal Power (MWt): 1650
4. Nameplate Ratigg (Gross MWe): 659 x 0.9 = 593
5. Design Electrical Rating (Net MWe): 530
6. Maximum Dependable Capacity (Gross MWe): 534
7. Maximum Dependable Capacity (Net MWe): 503
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	4,367.0	57,335.0
12. Hours Reactor Was Critical	720.0	4,235.9	45,444.3
13. Reactor Reserve Shutdown Hours	.0	.0	5,531.9
14. Hours Generator On-Line	720.0	4,230.8	44,354.4
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,128,884	6,834,962	68,913,936
17. Gross Electrical Energy (MWH)	360,900	2,114,389	22,268,050
18. Net Electrical Energy (MWH)	318,484	1,981,867	20,821,119
19. Unit Service Factor	100.0	96.9	77.4
20. Unit Availability Factor	100.0	96.9	77.4
21. Unit Capacity Factor (MDC Net)	87.9	90.2	72.2
22. Unit Capacity Factor (DER Net)	83.5	85.6	68.5
23. Unit Forced Outage Rate	.0	.0	12.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUELING, AUGUST 15, 1980, - 8 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	06/29/80	F	.0	H	5				REDUCED POWER TO 50% TO DO THE MONTHLY TURBINE VALVES TEST. A RESULTING POWER DISTRIBUTION IMBALANCE FORCED OPERATION AT OR BELOW 50% FOR THE NEXT 24 HOURS.

SUMMARY

THE UNIT RAN NORMALLY FOR THE REPORTING PERIOD WITH A POWER REDUCTION OF 50% ON 06/29/80 FOR TURBINE VALVES TEST.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

PRAIRIE ISLAND 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Minnesota/Goodhue 28 miles SE of Minneapolis
2.	Electric Reliability Council	Mid-Continent Area Reliability Coordination Agreement
3.	Type of Reactor	PWR
4.	Power Level (MWT)	1650
5.	Design Electrical Rating (MWE Net)	530
6.	Date of Initial Criticality	December 17, 1974
7.	Date of Commercial Operation	December 21, 1974
8.	Condenser Cooling Method	Cooling Towers
9.	Condenser Cooling Water Source	Mississippi River

Utility & Contractor Information

10.	Licensee	Northern States Power Company
11.	Corporate Address	414 Nicollet Mall - 8th Floor Minneapolis, Minnesota 55401
12.	Corporate Contact	G. H. Neils (Oper.) L. O. Mayer (Lic.)
13.	Architect/Engineer	Pioneer Services & Eng. Co.
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Northern States Power Co.

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	W. S. Little
19.	IE Resident Inspector	C. D. Feierabend 612-388-8209
20.	Licensing Project Manager	R. Martin
21.	Docket Number	50-306
22.	License Number/Date of Issue	DPR-60 issued 10/29/74
23.	Public Document Room	Environmental Conservation Library Minneapolis Public Library 300 Nicollet Mall Minneapolis, Minnesota 55401

INSPECTION STATUS

ENFORCEMENT STATUS

YOUR RESPONSES TO IE BULLETIN NO. 78-05, DATED MAY 30, 1978 AND AUGUST 22, 1979, COMMIT THAT THE SPECIAL INSPECTIONS OF PLUNGER ARM CLEARANCE ON MOTOR STARTERS ON SAFETY RELATED SYSTEMS WOULD BE COMPLETED DURING THE NEXT REFUELING OUTAGE. BOTH UNITS HAVE SUBSEQUENTLY COMPLETED REFUELING OUTAGES. CONTRARY TO THE ABOVE, THE SPECIAL INSPECTIONS HAD NOT BEEN COMPLETED FOR THE MOTOR STARTERS ASSOCIATED WITH THE POWER OPERATED RELIEF VALVE ISOLATION VALVES FOR EITHER UNIT.
(8007 DEVIATION)

THE CORRECTED VISION OF SOME SECURITY FORCE PERSONNEL DID NOT MEET ESTABLISHED STANDARDS.
(8008 DEFICIENCY)

THE INTRUSION DETECTION SYSTEM DID NOT DETECT PENETRATION OF THE BOUNDARY AT SOME LOCATIONS.
(8008 DEFICIENCY)

SOME VITAL AREA PHYSICAL BARRIERS WERE NOT PROPERLY MAINTAINED.
(8008 INFRACTION)

AN UNATTENDED DESIGNATED VEHICLE IN THE PROTECTED AREA WAS NOT POSITIVELY CONTROLLED.
(8008 DEFICIENCY)

SOME ALARM DEVICES AND MONITORING SYSTEMS WERE NOT TAMPER-INDICATING.
(8008 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

Operating routinely.

+LAST IE SITE INSPECTION DATE: May 1-31, 1980

+ INSPECTION REPORT NO: 80-10

SUMMARY

INSPECTION SUMMARY

+ Meeting on February 26, (80-05): Discussed plant performance and enforcement history for the Prairie Island Plant, including the enforcement action resulting from inspection conducted in January 1980. Also discussed the implementation of the resident inspector program at the Prairie Island Plant.

+ Inspection on May 1-31, (80-10): Routine, resident inspection of plant operation, maintenance, surveillance, security, radiation protection, followup of licensee reported events, followup of IE Circulars and followup of licensee actions related to TMI lessons learned commitments. The inspection involved 87 inspector hours onsite by two NRC inspectors. No items of noncompliance or deviations were identified.

REPORTS RECEIVED FROM LICENSEE

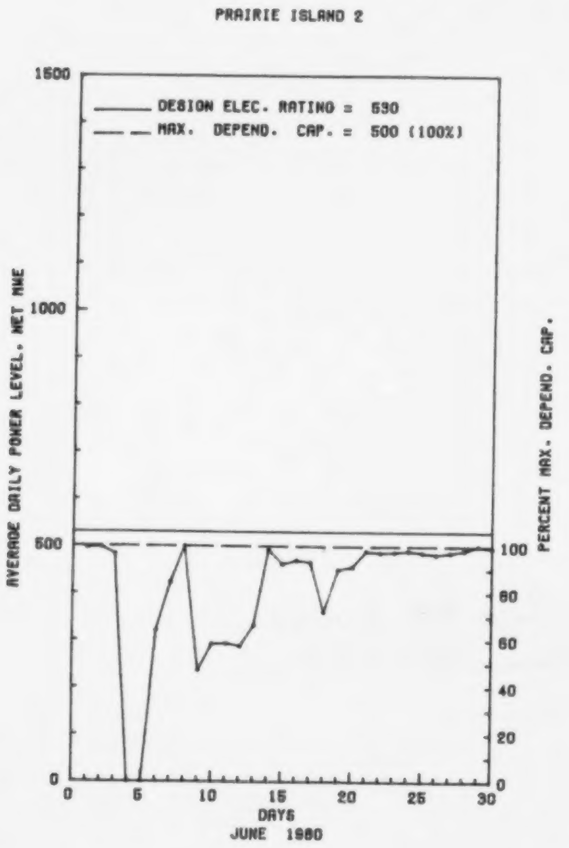
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT	PUBLIC ANNOUNCEMENT (Yes or no, date, and by whom)
	NONE				

REVIEWED BY OMPA LOEB: E Hill **DATE:** 8 4 80

REVIEWED BY NRR DL: R Martin **DATE:** 8/5/80

PRAIRIE ISLAND 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-306
2. Utility Contact: A. A. Hunstad (612) 388-4767
3. Licensed Thermal Power (MWt): 1650
4. Nameplate Rating (Gross MWe): 659 x 0.9 = 593
5. Design Electrical Rating (Net MWe): 530
6. Maximum Dependable Capacity (Gross MWe): 531
7. Maximum Dependable Capacity (Net MWe): 500
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	4,367.0	48,453.0
12. Hours Reactor Was Critical	679.5	3,064.6	42,193.6
13. Reactor Reserve Shutdown Hours	40.5	82.9	1,488.9
14. Hours Generator On-Line	667.2	2,997.9	41,398.9
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,003,265	4,681,932	64,354,435
17. Gross Electrical Energy (MWH)	317,420	1,508,260	20,562,390
18. Net Electrical Energy (MWH)	295,518	1,406,745	19,245,377
19. Unit Service Factor	92.7	68.6	85.4
20. Unit Availability Factor	92.7	68.6	85.4
21. Unit Capacity Factor (MDC Net)	82.1	64.4	79.4
22. Unit Capacity Factor (DER Net)	77.4	60.8	74.9
23. Unit Forced Outage Rate	.0	.4	4.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
REFUELING, JANUARY, 1981, - 6 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	06/04/80	S	49.2	B	3				SHUTDOWN TO RETURN 2M TRANSFORMER TO SERVICE AND PERFORM MAINTENANCE ON THE TURBINE OIL SYSTEM.
	06/06/80	S	3.6	H	1				MANUAL TURBINE TRIP BECAUSE OF HIGH VIBRATION.
	06/09/80	F	.0	H	5				POWER WAS REDUCED TO 65% FROM 06/09/80 TO 06/13/80 DUE TO DOWNED LINES FROM SUBSTATION.

SUMMARY

THE UNIT WAS SHUTDOWN ON 06/04/80 FOR MAINTENANCE ON TURBINE SYSTEM. A MANUAL TURBINE TRIP OCCURRED ON 06/06/80. THE UNIT RAN NORMALLY AFTERWORDS WITH A POWER REDUCTION ON 06/09/80 DUE TO DOWN LINES.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

QUAD CITIES 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Illinois/Rock Island 20 miles NE of Moline
2.	Electric Reliability Council	Mid-America Interpool Network
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2511
5.	Design Electrical Rating (MWE Net)	789
6.	Date of Initial Criticality	October 18, 1971
7.	Date of Commercial Operation	February 18, 1973
8.	Condenser Cooling Method	Spray Canal
9.	Condenser Cooling Water Source	Mississippi River

Utility & Contractor Information

10.	Licensee	Commonwealth Edison Co.
11.	Corporate Address	P. O. Box 767 Chicago, Illinois 60690
12.	Corporate Contact	D. L. Peoples
13.	Architect/Engineer	Sargent and Lundy
14.	Nuclear Steam Supply System	GE
15.	Constructor	United Engineers & Constructors, Inc.

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	R. Spessard
19.	IE Resident Inspector	N. J. Chrissotimus (309) 654-2227
20.	Licensing Project Manager	R. B. Bevan
21.	Docket Number	50-254
22.	License Number/Date of Issue	DPR-29 issued 10/1/71
23.	Public Document Room	Moline Public Library 504 17th Street Moline, Illinois 61265

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

Unit in coastdown mode at end of life. Refueling outage scheduled for September 1, 1980.

LAST IE SITE INSPECTION DATE: March 27 through May 7, 1980

INSPECTION REPORT NO: 80-10

SUMMARY

INSPECTION SUMMARY

+ No inspections conducted.

REPORTS RECEIVED FROM LICENSEE

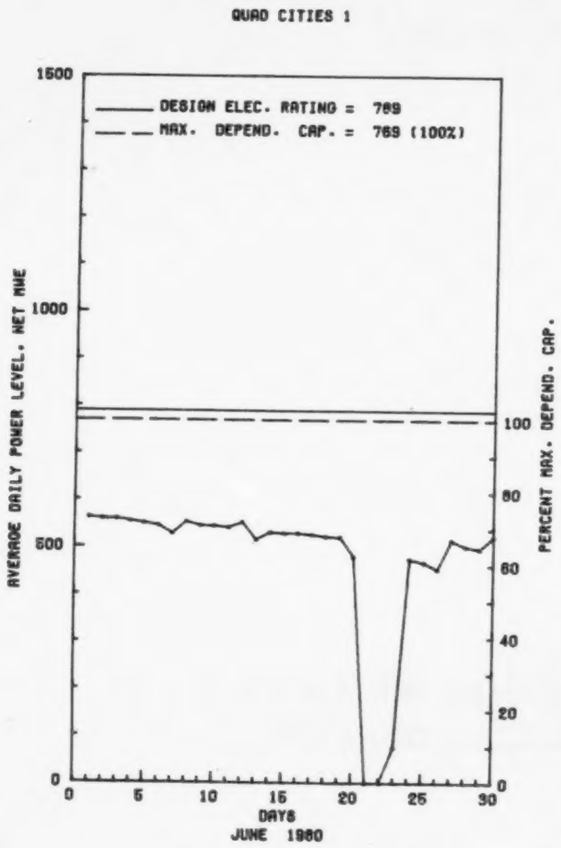
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-11/ 03L-0	04/29/80	05/27/80	While attempting to start the 1B reactor water cleanup pump for maintenance the main feed breaker tripped.
80-12/ 03L-0	04/29/80	05/23/80	During steady state power operation, the 1A reactor Recirc Motor generator set tripped.
80-15/ 03L-0	05/11/80	06/09/80	During procedure QOS 201-1, relief valve 1-203-3B failed to open (TS 3.5.D.2).

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 8-4-80

REVIEWED BY NRR DL: RBBevan **DATE:** 8/5/80

QUAD CITIES 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-254
2. Utility Contact: T. HAFERA (309) 654-2241 X176
3. Licensed Thermal Power (MWt): 2511
4. Nameplate Rating (Gross MWe): 920 x 0.9 = 828
5. Design Electrical Rating (Net MWe): 789
6. Maximum Dependable Capacity (Gross MWe): 813
7. Maximum Dependable Capacity (Net MWe): 769
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	4,367.0	71,327.0
12. Hours Reactor Was Critical	710.4	4,357.4	50,071.5
13. Reactor Reserve Shutdown Hours	.0	.0	3,421.9
14. Hours Generator On-Line	654.4	4,289.2	56,330.1
15. Unit Reserve Shutdown Hours	.0	.0	909.2
16. Gross Thermal Energy (MWH)	1,192,667	9,291,377	113,835,358
17. Gross Electrical Energy (MWH)	378,942	3,017,325	36,640,379
18. Net Electrical Energy (MWH)	340,669	2,815,289	34,230,825
19. Unit Service Factor	90.9	98.2	79.0
20. Unit Availability Factor	90.9	98.2	80.2
21. Unit Capacity Factor (MDC Net)	61.5	83.8	62.4
22. Unit Capacity Factor (DER Net)	60.0	81.7	60.8
23. Unit Forced Outage Rate	9.1	1.8	7.4

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUELING, 09/01/80
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
8	06/20/80	F	65.6	A	3				REACTOR SCRAM DUE TO LOW MAIN CONDENSER VACUUM.
9	06/25/80	F	.0	A	5				LOAD REDUCTION DUE TO A SMALL LEAK ON THE 1B REACTOR FEED PUMP.

SUMMARY

THE UNIT OPERATED AT REDUCED POWER UNTIL 06/07/80 WHEN LOAD WAS REDUCED FOR CONDENSATE FLOW REVERSAL. A SCRAM DUE TO LOW CONDENSER VACUUM OCCURRED ON 06/20/80. REDUCTIONS OCCURRED ON 06/24/80 AND 06/25/80 FOR CONTROL ROD PATTERN CHANGE AND REACTOR FEED PUMP LEAKAGE.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

QUAD CITIES 2

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Illinois/Rock Island 20 miles NE of Moline
2.	Electric Reliability Council	Mid-America Interpool Network
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2511
5.	Design Electrical Rating (MWE Net)	789
6.	Date of Initial Criticality	April 26, 1972
7.	Date of Commercial Operation	March 10, 1973
8.	Condenser Cooling Method	Spray Canal
9.	Condenser Cooling Water Source	Mississippi River

Utility & Contractor Information

10.	Licensee	Commonwealth Edison Co.
11.	Corporate Address	P. O. Box 767 Chicago, Illinois 60690
12.	Corporate Contact	D. L. Peoples
13.	Architect/Engineer	Sargent & Lundy
14.	Nuclear Steam Supply System	GE
15.	Constructor	United Engineers & Constructors, Inc.

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	R. Spessard
19.	IE Resident Inspector	N. J. Chrissotimus (309) 654-2227
20.	Licensing Project Manager	R. B. Bevan
21.	Docket Number	50-265
22.	License Number/Date of Issue	DPR-30 issued 3/21/72
23.	Public Document Room	Moline Public Library 504 17th Street Moline, Illinois 61265

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

Plant operating routinely.

LAST IE SITE INSPECTION DATE: March 27 through May 7, 1980

INSPECTION REPORT NO: 80-13

SUMMARY

INSPECTION SUMMARY

+ No inspections conducted.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

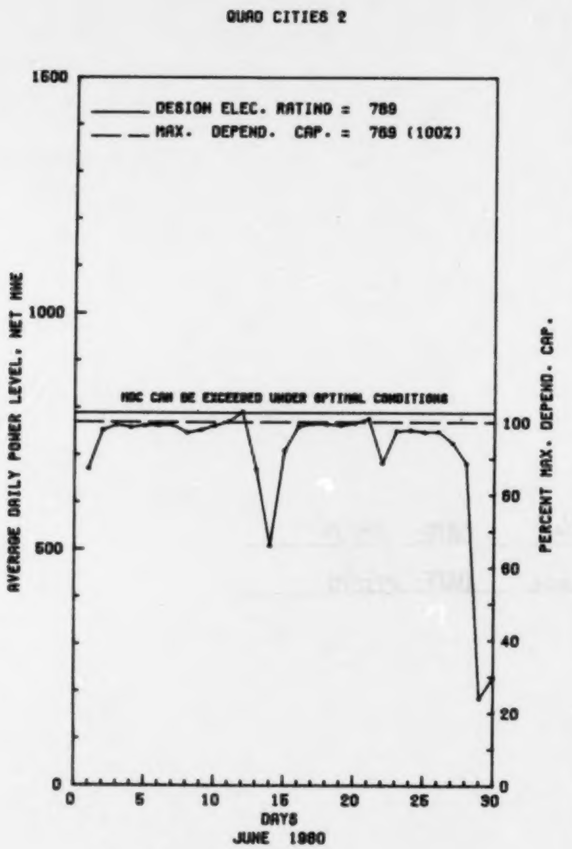
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-10/03L-0	05/17/80	06/10/80	While performing quarterly MSIV closure timing surveillance, proced. QOS 250-4, MSIV AO 2-203-1C closed in 2.5 seconds MSIV AO 2-203-2D closed in 9.1 seconds.
80-12/03L-0	04/28/80	05/27/80	During startup reactor water analysis indicated conductivity was 13 mmhos/cm, which exceeded TS limit.
80-13/03L-0	04/28/80	05/27/80	While performing QOS 1000-3, the RHR to suppression chamber test and cooling valve MO 2-1001-36A failed to close.

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 8-4-80

REVIEWED BY NRR DL: RBBevan **DATE:** 8/5/80

QUAD CITIES 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-265
2. Utility Contact: T. HAFERA (309) 654-2241 X176
3. Licensed Thermal Power (MWt): 2511
4. Nameplate Rating (Gross MWe): 920 x 0.9 = 828
5. Design Electrical Rating (Net MWe): 789
6. Maximum Dependable Capacity (Gross MWe): 813
7. Maximum Dependable Capacity (Net MWe): 769
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	4,367.0	70,437.0
12. Hours Reactor Was Critical	709.3	1,582.7	54,711.6
13. Reactor Reserve Shutdown Hours	.0	.0	2,985.8
14. Hours Generator On-Line	695.2	1,479.4	52,271.2
15. Unit Reserve Shutdown Hours	.0	.0	702.9
16. Gross Thermal Energy (MWH)	1,652,033	3,267,455	106,283,564
17. Gross Electrical Energy (MWH)	528,727	1,038,218	33,901,268
18. Net Electrical Energy (MWH)	505,246	947,821	31,690,346
19. Unit Service Factor	96.6	33.9	74.2
20. Unit Availability Factor	96.6	33.9	75.2
21. Unit Capacity Factor (MDC Net)	91.3	28.2	58.5
22. Unit Capacity Factor (DER Net)	88.9	27.5	57.0
23. Unit Forced Outage Rate	3.4	13.0	9.4

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
7	06/09/80	F	.0	A	5				LOAD REDUCTION DUE TO OUTPUT BREAKER ON A REACTOR PROTECTION MG SET TRIP.
8	06/13/80	F	.0	A	5				LOAD REDUCTION DUE TO SEAL WATER LINE LEAK ON THE 2A REACTOR FEED PUMP.
9	06/29/80	F	24.8	A	1				SHUTDOWN TO REPAIR STEAM LEAK ON TURBINE CONTROL VALVE.

SUMMARY

UNIT TWO AT NEARLY FULL POWER IN JUNE. THERE WERE REDUCTIONS FOR CONDENSER REVERSAL, HIGH CONDENSER BACK PRESSURE, TURBINE TESTING, FEED PUMP SEAL LEAKAGE AND BREAKER FAILURE. THE REACTOR WAS SHUTDOWN ON 06/29/80 TO REPAIR A STEAM LEAK ON A TURBINE CONTROL VALVE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

RANCHO SECO

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	California/Sacramento 25 miles SE of Sacramento
2.	Electric Reliability Council	Western Systems Coordinating Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2772
5.	Design Electrical Rating (MWE Net)	918
6.	Date of Initial Criticality	September 16, 1974
7.	Date of Commercial Operation	April 17, 1975 (92.6% of full power)
8.	Condenser Cooling Method	Closed Cycle
9.	Condenser Cooling Water Source	Natural Draft Cooling Towers - Folsom Canal

Utility & Contractor Information

10.	Licensee	Sacramento Municipal Utility District
11.	Corporate Address	6201 South Street P. O. Box 15830 Sacramento, California
12.	Corporate Contact	J. J. Mattimoe Assistant General Manager, Chief Engineer
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	B&W
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	V
17.	IE Branch Chief	J. L. Crews
18.	IE Section Chief	B. H. Faulkenberry
19.	IE Resident Inspector	H. L. Canter (209) 748-2791
20.	Licensing Project Manager	D. J. Garner
21.	Docket Number	50-312
22.	License Number/Date of Issue	DPR-54, 8/16/74
23.	Public Document Room	Business and Municipal Department Sacramento City-County Library 828 I Street Sacramento, California 95814

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

SYSTEMS AND COMPONENT PROBLEMS:

None

FACILITY ITEMS (PLANS AND PROCEDURES):

None.

MANAGERIAL ITEMS:

Awaiting decision of ASLB.

PLANT STATUS:

Operation at or near full power.

LAST IE SITE INSPECTION DATE: 06/02-30/80+

INSPECTION REPORT NO. 50-312/80-20+

SUMMARY

INSPECTION SUMMARY

+ Inspection on May 5-9, 1980 (Report No. 50-312/80-13) Areas Inspected: Routine, unannounced inspection of the document control program; Onsite Review Committee operations; the reactor operator requalification program; and followup on an IE Bulletin. The inspection involved 36 inspector hours onsite by one NRC inspector.

Results: No items of noncompliance or deviations were identified.

+ Inspection On April 14-26, 1980 (Report No. 50-312/80-15) PAT Inspection; report being prepared.

+ Inspection on May 1-30, 1980 (Report No. 50-312/80-17) Areas Inspected: Routine inspections of operations; plant trips; followup on items of noncompliance; followup on Headquarters' requests; and independent inspection effort. The inspection involved 76 inspector hours onsite by the senior Resident Inspector.

Results: No items of noncompliance or deviations were identified.

+ Inspection on June 9-13, 1980 (Report No. 50-312/80-18) Areas Inspected: Routine, unannounced inspection of operator requalification program; procedures for control of test and measuring equipment; overall facility procedure status; followup of proposed plant modifications; and witnessing of an emergency drill. The inspection involved 34 inspector hours onsite by one NRC inspector.

Results: Of the five areas inspected, no items of noncompliance or deviations were found in three areas. Two items of noncompliance were found in two areas (Infraction - Absence of documentation showing that an operator had completed the approved requalification program as certified; and Infraction - Failure to provide list of items requiring calibration and identify calibration frequency).

+ Inspection on June 9-13, 1980 (Report No. 50-312/80-19) Areas Inspected: Routine, unannounced inspection by a Regional based inspector of the radiation protection program during operation including: training; observation of the annual emergency drill; and tours of the controlled area. The inspection involved 36 inspector hours onsite by one NRC inspector.

Results: Of the three areas inspected, no items of noncompliance or deviations were found in two areas. Two items of noncompliance were found in one area (Infraction - Failure to instruct workers, 10 CFR 19.12; Infraction - Failure to maintain a retraining program, Technical Specification 6.4.1).

+ Inspection on June 2-30, 1980 (Report No. 50-312/80-20) Report being prepared; to be reported next month.

NRC POWER RESTRICTIONS: None

REPORTS RECEIVED FROM LICENSEE

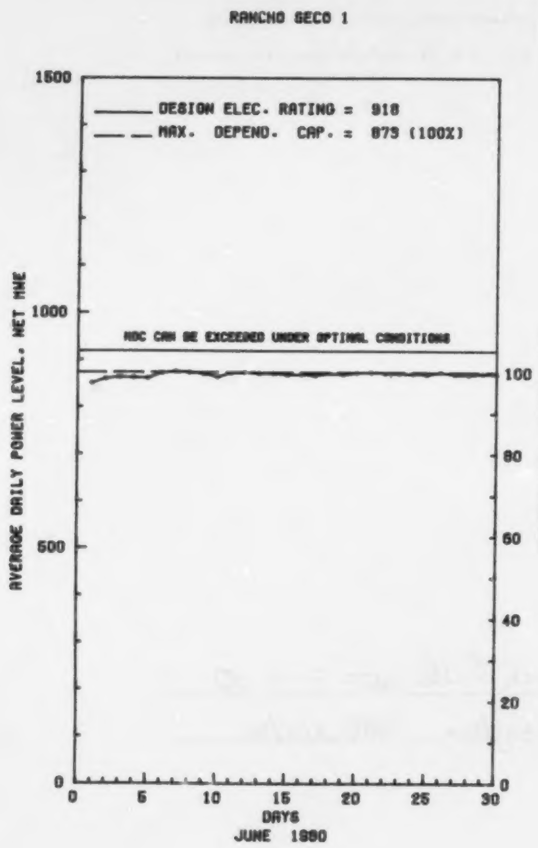
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
	NONE		

REVIEWED BY OMPA LOEB: P E Bole **DATE:** 8/4/80

REVIEWED BY NRR DL: D J Garner **DATE:** 8/5/80

RANCHO SECO

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-312
2. Utility Contact: R. W. Colombo (916) 452-3211
3. Licensed Thermal Power (MWt): 2772
4. Nameplate Rating (Gross MWe): 1070 x 0.9 = 963
5. Design Electrical Rating (Net MWe): 918
6. Maximum Dependable Capacity (Gross MWe): 917
7. Maximum Dependable Capacity (Net MWe): 873
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	4,367.0	45,624.0
12. Hours Reactor Was Critical	720.0	1,514.0	27,587.5
13. Reactor Reserve Shutdown Hours	.0	.0	3,975.1
14. Hours Generator On-Line	720.0	1,429.6	26,370.9
15. Unit Reserve Shutdown Hours	.0	.0	1,210.2
16. Gross Thermal Energy (MWH)	1,902,171	3,587,222	66,620,922
17. Gross Electrical Energy (MWH)	638,818	1,196,626	22,481,162
18. Net Electrical Energy (MWH)	607,076	1,123,942	21,234,832
19. Unit Service Factor	100.0	32.7	57.6
20. Unit Availability Factor	100.0	32.7	60.5
21. Unit Capacity Factor (MDC Net)	96.6	29.5	53.3
22. Unit Capacity Factor (DER Net)	91.8	28.0	50.7
23. Unit Forced Outage Rate	.0	5.3	31.0

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	NONE								

SUMMARY

THE PLANT RAN AT 96% POWER LEVEL FOR THE REPORTING PERIOD.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

ROBINSON 2

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	South Carolina/Darlington 5miles NW of Hartsville
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2300
5.	Design Electrical Rating (MWE Net)	700
6.	Date of Initial Criticality	September 20, 1970
7.	Date of Commercial Operation	March 7, 1971
8.	Condenser Cooling Method	Recirculation from impounded lake
9.	Condenser Cooling Water Source	Robinson Impoundment

Utility & Contractor Information

10.	Licensee	Carolina Power and Light Company
11.	Corporate Address	411 Fayetteville Street Raleigh, North Carolina
12.	Corporate Contact	J. A. Jones Executive Vice President
13.	Architect/Engineer	Ebasco
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Ebasco

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R.C. Lewis
18.	IE Section Chief	P. J. Kellogg
19.	IE Resident Inspector	R. Sauer
20.	Licensing Project Manager	D. Neighbors
21.	Docket Number	50-261
22.	License Number/Date of Issue	DPR-23, 7/31/70 (5 Mwt), Full power authorization issued 9/23/70
23.	Public Document Room	Hartsville Memorial Library Home and Fifth Avenue Hartsville, South Carolina 29550

R-4

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

- SCHEDULED TO SHUTDOWN FOR REFUELING, JULY 1980.
- LAST IE SITE INSPECTION DATE: JUNE 10-12, 1980
- INSPECTION REPORT NO: 50-261/80-12

SUMMARY

INSPECTION SUMMARY

- INSPECTION JUNE 10-12 (80-12): INCLUDED REVIEW OF SECURITY ORGANIZATION. THE INSPECTION INVOLVED 30 INSPECTOR-HOURS ON SITE BY TWO NRC INSPECTORS. THE INSPECTION WAS BEGUN DURING AN OFFSHIFT PERIOD; 10 INSPECTION HOURS WERE ACCOMPLISHED DURING OFFSHIFT PERIODS. WITHIN THE AREA INSPECTED, THERE WERE NO ITEMS OF NONCOMPLIANCE IDENTIFIED.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

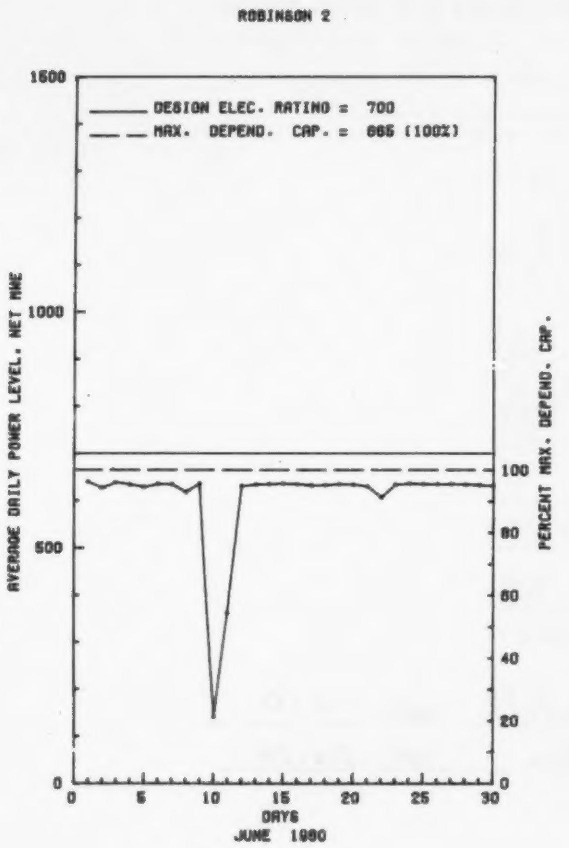
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-010/03L-0	04/25/80	05/22/80	SHIFT OF A HIGH STEAM FLOW PROTECTION MODULE OUTPUT IN THE NON-CONSERVATIVE DIRECTION.
80-011/03L-0	04/24/80	05/23/80	LOSS OF SOURCE RANGE INDICATION WHEN BOTH CHANNELS WERE TESTED SIMULTANEOUSLY.
80-012/03L-0	05/06/80	06/05/80	OPERATION WITH A FAULTY NIS POWER RANGE DETECTOR CAUSED THREE RUNBACKS TO OCCUR IN ONE HOUR PERIOD.
80-013/03L-0	05/17/80	05/30/80	RADIAL TILT OUTSIDE 2% DURING OPERATION AT 40% AND POWER RANGE TRIP SETPOINTS WERE NOT REDUCED.

REVIEWED BY OMPA LOEB: Earl Hill **DATE:** 8-4 80

REVIEWED BY NRR DL: Don Neighbors **DATE:** 8/5/80

ROBINSON 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-261
2. Utility Contact: M. L. Watford (803) 383-4524
3. Licensed Thermal Power (MWt): 2300
4. Nameplate Rating (Gross MWe): 854 x 0.9 = 769
5. Design Electrical Rating (Net MWe): 700
6. Maximum Dependable Capacity (Gross MWe): 700
7. Maximum Dependable Capacity (Net MWe): 665
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): 2200 MWT
10. Reasons for Restrictions, If Any:
EXCESSIVE MOISTURE CARRY-OVER TO H.P. TURBINE.

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	4,367.0	81,749.0
12. Hours Reactor Was Critical	718.1	3,592.9	63,799.8
13. Reactor Reserve Shutdown Hours	1.8	202.4	921.7
14. Hours Generator On-Line	712.7	3,573.3	62,264.2
15. Unit Reserve Shutdown Hours	.0	.0	23.2
16. Gross Thermal Energy (MWH)	1,486,150	7,144,149	127,694,478
17. Gross Electrical Energy (MWH)	461,175	2,291,336	41,227,849
18. Net Electrical Energy (MWH)	437,062	2,166,179	39,071,281
19. Unit Service Factor	99.0	81.8	76.2
20. Unit Availability Factor	99.0	81.8	76.2
21. Unit Capacity Factor (MDC Net)	91.3	74.6	71.9
22. Unit Capacity Factor (DER Net)	86.7	70.9	68.3
23. Unit Forced Outage Rate	1.0	18.2	13.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
REFUELING/MAINTENANCE, JULY - 7 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
06-01	06/10/80	F	2.4	A	3		HA	CKTBRK	A DEFECTIVE CONTACT (RELAY 152X) CAUSED THE TURBINE GOVERNOR AND INTERCEPT VALVES TO UNEXPECTEDLY CLOSE RESULTING IN A REACTOR TRIP DUE TO HIGH PRESSURIZER PRESSURE.
06-02	06/10/80	F	4.9	A	1		HC	PUMPXX	"A" CONDENSER VACUUM PUMP WAS INOPERABLE DUE TO A MOTOR GROUND AND "B" PUMP LOST SEAL WATER FLOW. THE UNIT WAS SEPARATED FROM THE SYSTEM AS CONDENSER VACUUM CONTINUED TO DROP.

SUMMARY

THE UNIT OPERATED NORMALLY FOR THE REPORTING PERIOD WITH A BRIEF OUTAGE ON 06/10/80 DUE TO SCRAMS CAUSED BY TURBINE AND CONDENSER VACUUM PUMP PROBLEMS.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

SALEM 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	New Jersey/Salem 20 miles S of Wilmington, Del.
2.	Electric Reliability Council	Mid-Atlantic Area Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	3338
5.	Design Electrical Rating (MWE Net)	1090
6.	Date of Initial Criticality	December 11, 1976
7.	Date of Commercial Operation	June 30, 1977
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Delaware River

Utility & Contractor Information

10.	Licensee	Public Service Electric & Gas Co.
11.	Corporate Address	80 Park Place Newark, New Jersey 07101
12.	Corporate Contact	F. P. Librizzi General Manager - Electric Prod.
13.	Architect/Engineer	PSE&G
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	UE&C

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	IE Section Chief	R. R. Keimig
19.	IE Resident Inspector	L. J. Norrholm (609-935-3850) (FTS 341-3000)
20.	Licensing Project Manager	W. Ross
21.	Docket Number	50-272
22.	License Number/Date of Issue	DPR-70, 8/13/76
23.	Public Document Room	Salem Free Public Library 112 West Broadway Salem, New Jersey 08079

INSPECTION STATUS

ENFORCEMENT STATUS

10 CFR 50, APP B, CRIT V, STATES IN PART, "ACTIVITIES AFFECTING QUALITY SHALL BE PRECRIBED BY DOCUMENTED INSTRUCTIONS, PROCEDURES...& SHALL BE ACCOMPLISHED IN ACCORDANCE WITH THESE INSTRUCTIONS, PROCEDURES...."
(8004 INFRACTION)

10 CFR 50, APP B, CRIT. V STATES IN PART "ACTIVITIES AFFECTING QUALITY SHALL BE PRESCRIBED BY DOCUMENTED INSTRUCTIONS, PROCEDURES ... OF A TYPE APPROPRIATE TO THE CIRCUMSTANCES ...." FSAR AMENDMENT 43, SECTION D.2, IDENTIFYING STRUCTURES, SYSTEMS & COMPONENTS COVERED BY THE QA PROGRAM ...." CONTRARY TO THE ABOVE, AS OF 2/13/80, THE LICENSEE HAD NOT DEVELOPED MEL'S FOR EITHER UNIT. FSAR AMENDMENT 43, SECTION D.2 STATES IN PART, "THESE ACTIVITIES SHALL BE PERFORMED IN COMPLIANCE WITH ...(8) REG GUIDE 16.4 ...." CONTRARY TO THE ABOVE, AS OF 2/13/80, THE LICENSEE HAD NOT ESTABLISHED PROCEDURES THAT: ADDRESSED HOW USERS OF CONTROLLED DRAWINGS FROM THE STATION TECHNICAL DOCUMENT ROOM WOULD BE ALERTED THAT A GIVEN DRAWING WAS AFFECTED BY A RECENT MODIFICATION (8004 INFRACTION)

10 CFR 50, APP B, CRIT XVII, STATES IN PART, "...THE APPLICANT SHALL ESTABLISH REQUIREMENTS CONCERNING RECORD RETENTIONS, SUCH AS DURATION, LOCATION & ASSIGNED RESPONSIBILITIES."
(8004 DEFICIENCY)

CONTRARY TO 10 CFR 50, APPENDIX B, CRITERION XII, FEEDWATER FLOW INSTRUMENTS USED IN CALORIMETRIC CALCULATION WERE NOT PROPERLY CALIBRATED.
(8006 INFRACTION)

CONTRARY TO LICENSE DPR-70 CONDITION, STEADY STATE OPERATION WAS CONDUCTED AT INDICATED POWER LEVEL IN EXCESS OF 3338 MWT.
(8006 INFRACTION)

CONTRARY TO TS 4.1.3.1.1, INDIVIDUAL ROD POSITION WAS NOT RECORDED EVERY 4 HOURS WITH DEVIATION MONITOR INOPERABLE.
(8006 INFRACTION)

CONTRARY TO TS 4.2.2, QUADRANT POWER TILT RATIO WAS NOT DETERMINED EVERY 12 HOURS AS REQUIRED WITH THE DEVIATION ALARM INOPERABLE.
(8006 DEFICIENCY)

CONTRARY TO TS 6.8.1 & OPERATIONS DIRECTIVES MANUAL 2.4.5, MANUAL ISOLATION VALVE 14AF10 WAS SHUT, ISOLATING AUXILIARY FEEDWATER FLOW FROM #13 AFP TO #14 STEAM GENERATOR. NO ADMINISTRATIVE CONTROL WAS PROVIDED WHICH RECOGNIZED THIS SITUATION.
(8006 INFRACTION)

CONTRARY TO 10 CFR 50 APPENDIX J V.B. THE LICENSEE FAILED TO SUBMIT A CONTAINMENT LEAKAGE RATE TEST SUMMARY REPORT TO THE NRC WITHIN THREE MONTHS. THE TEST WAS COMPLETED ON AUGUST 13, 1979 AND THE REPORT WAS SUBMITTED ON MARCH 31, 1980. FURTHER, NOT ALL TYPE C LOCAL LEAK TESTS CONDUCTED WERE INCLUDED IN THE REPORT. ONE EXAMPLE IS A SPECIAL TEST OF VALVE 1SS107 ON MAY 11, 1978.
(8010 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

- Permanent weld repairs have been effected for the spent fuel pool liner leak.

FACILITY ITEMS (PLANS AND PROCEDURES):

- The next refueling outage is scheduled for September 1980.

PLANT STATUS:

- Unit is in normal operation.

- LAST IE SITE INSPECTION DATE: 6/30/80
- INSPECTION REPORT NO: 50-272/80-14

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJEC
80-022/03L	04/22/80	05/19/80	No. 11 containment fan coil unit inoperable due to failed coil on solenoid valve SV621.
80-023/03L	04/24/80	05/19/80	No. 12 containment fan coil unit inoperable due to failed flow transmitter
80-024/03L	04/25/80	05/20/80	No. 12 containment fan coil unit inoperable due to failure of solenoid valve SV624.
80-025/03L	05/16/80	06/12/80	Loss of containment integrity due to failed sample line penetration
80-026/03L	05/16/80	06/12/80	Steam generator low level trip setpoint out of spec

SUMMARY

INSPECTION SUMMARY

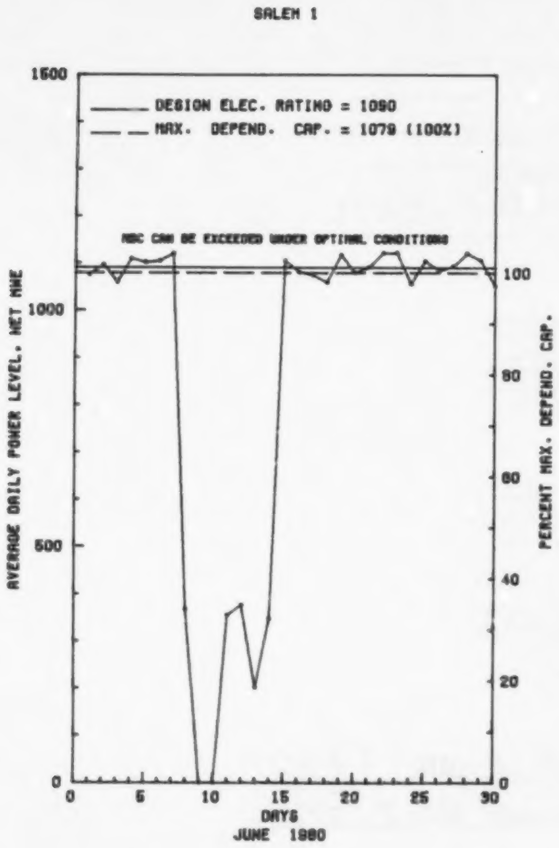
- 50-272/80-03 - Jan 28 - Feb 8: Special, announced Health Physics Appraisal inspection including a review of selected procedures and representative records, observed work practices, and interviewed personnel. Three items of noncompliance were identified: 1. Failure to follow radiation protection calibration procedures; 2. Failure to properly review and approve procedures; 3. Failure to label containers of radioactive material.
- 50-272/80-06 - Mar 30 - Apr 30: Routine inspections by the resident inspectors (67 hours) of plant operations including: tours of the facility; log and record reviews; IE Circulars; management meetings; implementation of containment isolation and control rod guidance; and, followup on previous inspection items. Five items of noncompliance were identified: 1. Partial isolation of Auxiliary Feedwater without taking appropriate action as required by Technical Specifications; 2. Exceeding licensed power level during steady state operation; 3. Failure to record individual rod position every four hours as required by Technical Specifications; 4. Failure to conduct monthly calibrations of feedwater flow instrumentation as required by 10 CFR 50, Appendix B, Criterion XII; 5. Failure to conduct quadrant power tilt ratio every twelve hours.
- 50-272/80-11 - May 1-31: Routine inspections by the resident inspectors (46 hours) of plant operations including tours of the facility; log and record reviews; review of licensee events; operational safety verification; review of strike contingency plans; and, followup on previous inspection items. No items of noncompliance were identified.
- 50-272/80-15 - May 19-21: Special inspection by one regional based inspector (24 hours) of Security Plan: Physical Barriers (protected area); Physical Barriers (vital area); Assessment Aids; Detection Aids; Alarm Stations and confirmation of actions taken to correct items of noncompliance reflected in Inspection Report 50/272/80-02. No items of noncompliance were identified.

REVIEWED BY OMPA LOEB: DATE: 8.4.80

REVIEWED BY NRR DL: WJRoss DATE: 8-5-80

SALEM 1

AVERAGE DAILY POWER LEVEL(MWe)



OPERATING STATUS

1. Docket Number: 50-272
2. Utility Contact: L. K. Miller (609) 365-7000
3. Licensed Thermal Power (MWt): 3338
4. Nameplate Rating (Gross MWe): 1300 x 0.9 = 1170
5. Design Electrical Rating (Net MWe): 1090
6. Maximum Dependable Capacity (Gross MWe): 1124
7. Maximum Dependable Capacity (Net MWe): 1079
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	4,367.0	26,328.0
12. Hours Reactor Was Critical	645.4	4,177.9	14,266.2
13. Reactor Reserve Shutdown Hours	.0	.0	22.7
14. Hours Generator On-Line	632.3	4,058.0	13,589.1
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,983,626	12,534,328	40,070,624
17. Gross Electrical Energy (MWH)	671,320	4,254,340	13,421,840
18. Net Electrical Energy (MWH)	640,951	4,071,530	12,701,120
19. Unit Service Factor	87.8	92.9	51.6
20. Unit Availability Factor	87.8	92.9	51.6
21. Unit Capacity Factor (MDC Net)	82.5	86.4	44.7
22. Unit Capacity Factor (DER Net)	81.7	85.5	44.3
23. Unit Forced Outage Rate	12.2	7.1	37.2

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUELING - 09/20/80 - 12/07/80.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-120	06/08/80	F	29.2	A	3		CH	INSTRU	INSTRUMENTATION FAILURE DUE TO LIGHTNING (S/G CHANNEL PRESSURE CONTROL).
80-122	06/09/80	F	39.4	A	4		AA	BLOWER	REPAIR #11 FAN COIL UNIT.
80-124	06/11/80	F	.0	B	5		CB	ZZZZZZ	REACTOR COOLANT SYSTEM LEAK TEST.
80-128	06/12/80	F	4.8	H	3		IA	INSTRU	TECHNICIAN ERROR DURING FUNCTIONAL TEST OF N-43 - INSTRUMENTATION CHANNEL.
80-130	06/13/80	F	8.9	H	3		EB	CKTBRK	1A VITAL 4160 VITAL BUS TRIP.
80-132	06/14/80	F	5.4	A	3		CH	PUMPXX	#12 STEAM GENERATOR PUMP TRIPPED DUE TO LOW LEVEL IN GENERATOR.
80-134	06/15/80	F	.0	A	5		HF	FILTER	#12 B CIRC. WATER PUMP OUT OF SERVICE DUE TO BROKEN CHAIN DRIVE ON TRAVELING SCREEN.
80-136	06/25/80	F	.0	A	5		HF	FILTER	HIGH DIFFERENTIAL PRESSURE ON #12A CIRCULATOR.

SUMMARY

THE UNIT RAN AT 100% POWER FROM 06/01/80 TO 06/07/80 AND FROM 06/16/80 TO THE END OF THE REPORTING PERIOD. ON 06/08/80, THE CONTAINMENT BUILDING WAS STRUCK BY LIGHTNING, RESULTING IN SCRAM DUE TO INSTRUMENTATION FAILURE. THE OUTAGE EXTENDED TO 06/10/80 FOR MAINTENANCE. SEVERAL OTHER OUTAGES AND POWER REDUCTIONS OCCURRED DURING THE REMAINDER OF THE MONTH.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

SAN ONOFRE 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	California/San Diego 5 miles S of San Clemente
2.	Electric Reliability Council	Western Systems Coordinating Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	1347
5.	Design Electrical Rating (MWE Net)	436
6.	Date of Initial Criticality	June 14, 1967
7.	Date of Commercial Operation	January 1, 1968
8.	Condenser Cooling Method	Circulating sea water
9.	Condenser Cooling Water Source	Pacific Ocean

Utility & Contractor Information

10.	Licensee	Southern California Edison Co.
11.	Corporate Address	2244 Walnut Grove Avenue Rosemead, California 91770
12.	Corporate Contact	R. E. Woodbury V.P. & General Counsel
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	V
17.	IE Branch Chief	J. L. Crews
18.	IE Section Chief	B. H. Faulkenberry
19.	IE Resident Inspector	L. F. MILLER (714-492-2641)
20.	Licensing Project Manager	S. J. Nowicki
21.	Docket Number	50-206
22.	License Number/Date of Issue	DPR-13, 3/27/67
23.	Public Document Room	Mission Viejo Branch Library 24851 Chrisanta Drive Mission Viejo, California 92676

INSPECTION STATUS

ENFORCEMENT STATUS

TECHNICAL SPECIFICATION SECTION 6.8.1 REQUIRES THAT WRITTEN PROCEDURES AND ADMINISTRATIVE POLICIES SHALL BE ESTABLISHED, IMPLEMENTED AND MAINTAINED. PROCEDURE S-VII 1.4, REVISION 5, APRIL 1979, "ENTERING AND LEAVING STEAM GENERATORS," A) SECTION III B2 REQUIRES THAT AIRBORNE PARTICULATES AND IODINE LEVELS BE DETERMINED FOR EACH STEAM GENERATOR INVOLVED PRIOR TO ENTRY; AND B) SECTION III E2 REQUIRES THAT THE WORKER WHO ENTERS A STEAM GENERATOR STAND STILL AFTER EXITING AND THAT THE ASSIGNED RADIATION PROTECTION TECHNICIAN REMOVE ITEMS OF SPECIAL PROTECTIVE CLOTHING AND EQUIPMENT. CONTRARY TO A), ON APRIL 12, 1980 WORK INVOLVING ENTRY INTO STEAM GENERATORS "A", "B" AND "C" WAS PERFORMED AND AIRBORNE ACTIVITY SURVEYS WERE NOT PERFORMED FOR EACH GENERATOR INVOLVED; AND CONTRARY TO B), THE SPECIAL PROTECTIVE CLOTHING AND EQUIPMENT WAS REMOVED BY WORKERS OTHER THAN THE RADIATION PROTECTION TECHNICIAN ASSIGNED.
(8014 INFRACTION)

INSPECTION STATUS

SYSTEMS AND COMPONENT PROBLEMS:

Refueling outage commenced two days earlier than scheduled due to increase in primary to secondary (S/G tube) leakage (PNO-V-80-24).

+ Large number of tubes determined to need plugging (results of eddy current testing).

FACILITY ITEMS (PLANS AND PROCEDURES):

Refueling.

MANAGERIAL ITEMS

None.

PLANT STATUS:

Plant shut down for refueling outage.

LAST IE SITE INSPECTION DATE: 06/01-30/80+

INSPECTION REPORT NO: 50-206/80-19+

SUMMARY

INSPECTION SUMMARY

+ Inspection on March 31-April 3, 1980 (Report No. 50-206/80-10)
Areas Inspected: Routine, unannounced inspection of the licensee's advanced planning and preparation efforts related to the refueling outage; investigation of licensee action in connection with a possible item of noncompliance related to a shipment of spent resin; followup on IE Circular 80-03 concerning protection from toxic gas hazards; and followup to Licensee Event Report No. 80-09. The inspection involved 32 inspector hours onsite by one NRC inspector.

Results: No items of noncompliance or deviations were identified.

+ Inspection on May 28-30, 1980 (Report No. 50-206/80-15)
Areas Inspected: Routine, announced inspection of steam generator eddy current inspection activities associated with IE Bulletin 79-13; and licensee action on outstanding items of noncompliance and unresolved items. The inspection involved 18 inspector hours onsite by one NRC inspector.

Results: No items of noncompliance or deviations were identified.

+ Inspection on May 5-30, 1980 (Report No. 50-206/80-16)
Areas Inspected: Routine, resident inspection during long term shutdown; monthly surveillance observations; monthly maintenance observations; inspection of refueling activities; review of core power distribution limit procedures, shutdown margin procedures, and licensee responses to IE Bulletins; independent inspection; Licensee Event Reports; and followup on loss of salt water cooling event.

Results: Of the nine areas inspected, no items of noncompliance or deviations were found in seven areas. One item of noncompliance was found in one area (Deficiency - Failure to properly implement Operating Instruction S-3-1.3). One deviation was identified in one area (Failure to maintain adequate housekeeping and cleanliness control).

+ Inspection on May 19-30, 1980 (Report No. 50-206/80-17)
Report being prepared; to be reported next month.

+ Inspection of May 19-22, 1980 (Report No. 50-206/80-18)
Areas Inspected: Routine, unannounced inspection of fire protection modifications; surveillance of pipe support and restraint systems; IE Bulletin/Circular followup; and independent inspection effort. The inspection involved 23 inspector hours onsite by one NRC inspector.

Results: No items of noncompliance or deviations were identified.

+ Inspection on June 1-30, 1980 (Report No. 50-206/80-19)
Report being prepared; to be reported next month.

+ Inspection on June 16-19, 1980 (Report No. 50-206/80-20)
Areas Inspected: Routine, unannounced inspection of major maintenance, major surveillance, IE Bulletin and Circular followup, and independent inspection effort. The inspection involved 24 inspector hours onsite by one NRC inspector.

Results: No items of noncompliance or deviations were identified.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

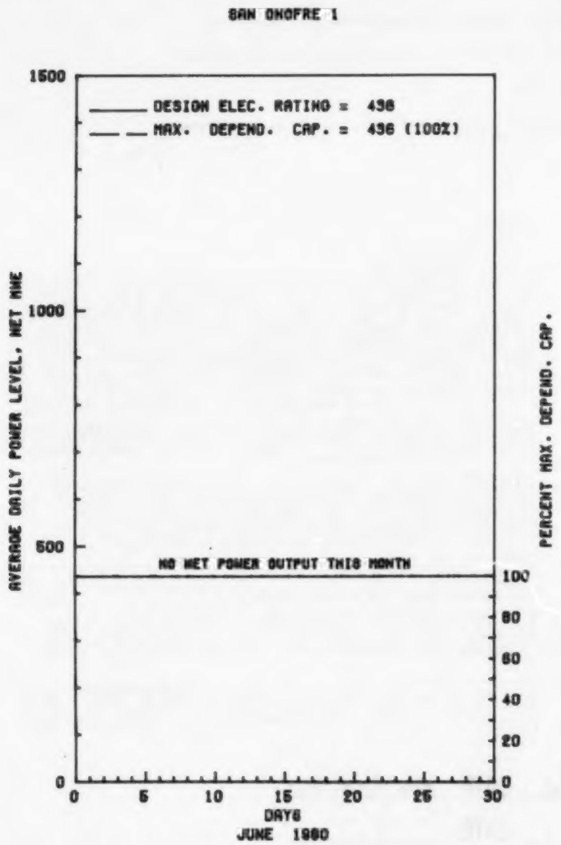
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-16/ 03L-0	04-20-80	05-20-80	Water leakage RV to ref. canal caused failures in source range instr.
80-17/ 01T-0	05-05-80	05-14-80	Erroneous boron concentration results due to contamination of PR1 MU tank.
80-18/ 01T-0	05-06-80	05-16-80	Incore flux thimble bent during refueling-degraded primary boundry.
80-19/ 01T-0	05-10-80	05-22-80	Missing U-bolt support RHR pump shaft cooling water return line (IEB 79-14).
80-20/ 01T-0	05-13-80	05-28-80	Surveillance functional testing found two failed snubbers.
80-21/ 01T-0	05-15-80	05-29-80	Feedwater system - deficient supports found during IEB 79-13 inspection.
80-22/ 01T-0	05-07-80	05-20-80	ISI revealed indication excess of allowable B RCP seal injection pipe.
80-24/ 01T-0	05-16-80	05-29-80	Radiographic exam of two main steam piping welds found two reportable indications.
80-25/ 99X-0	05-05-80	06-06-80	Linear indications found in welds in RCP flywheel bores.

REVIEWED BY OMPA LOEB: Earl Hill **DATE:** 8-4-80

REVIEWED BY NRR DL: S J Nowicki **DATE:** 8/5/80

SAN ONOFRE 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-206
2. Utility Contact: R. G. Moran (714) 492-7700
3. Licensed Thermal Power (MWt): 1347
4. Nameplate Rating (Gross MWe): 500 x 0.9 = 450
5. Design Electrical Rating (Net MWe): 436
6. Maximum Dependable Capacity (Gross MWe): 456
7. Maximum Dependable Capacity (Net MWe): 436
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	4,367.0	119,343.0
12. Hours Reactor Was Critical	.0	1,979.1	84,611.3
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	.0	1,961.6	81,119.0
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	2,552,787	104,084,261
17. Gross Electrical Energy (MWH)	0	855,000	35,526,434
18. Net Electrical Energy (MWH)	0	816,676	33,464,042
19. Unit Service Factor	.0	44.9	70.9
20. Unit Availability Factor	.0	44.9	70.9
21. Unit Capacity Factor (MDC Net)	.0	42.9	67.5
22. Unit Capacity Factor (DER Net)	.0	42.9	67.5
23. Unit Forced Outage Rate	.0	2.8	9.2

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 09/01/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
53	04/09/80	S	720.0	C	4		RC	FUELXX	REFUELING.

SUMMARY

THE UNIT WAS SHUTDOWN FOR REFUELING AND MAINTENANCE OUTAGE FOR THE REPORTING PERIOD.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

ST. LUCIE 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Florida/St. Lucie 12 miles SE of Ft. Pierce
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2560
5.	Design Electrical Rating (MWE Net)	802
6.	Date of Initial Criticality	April 22, 1976
7.	Date of Commercial Operation	December 21, 1976
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Atlantic Ocean

Utility & Contractor Information

10.	Licensee	Florida Power & Light Co.
11.	Corporate Address	9250 W. Flagler Street P. O. Box 529100 Miami, Florida 33152
12.	Corporate Contact	R. E. Uhrig, V.P.
13.	Architect/Engineer	Ebasco
14.	Nuclear Steam Supply System	Combustion Engineering
15.	Constructor	Ebasco

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. LEWIS
18.	IE Section Chief	R. A. Martin
19.	IE Resident Inspector	S. A. ELROD
20.	Licensing Project Manager	C. Nelson
21.	Docket Number	50-335
22.	License Number/Date of Issue	DPR-67, March 1, 1976
23.	Public Document Room	Indian River Junior College Library 3209 Virginia Avenue Ft. Pierce, Florida 33450

INSPECTION STATUS

ENFORCEMENT STATUS

FAILURE TO ESTABLISH, MAINTAIN AND FOLLOW ADEQUATE WRITTEN PROCEDURES. 2.790 INFORMATION.
(8004 INFRACTION)

FAILURE TO PROPERLY SEARCH PERSONNEL 2.790 INFORMATION
(8004 INFRACTION)

FAILURE TO PROPERLY CONTROL ACCESS TO A VITAL AREA - 2.790D INFORMATION.
(8004 INFRACTION)

FAILURE TO MAINTAIN A VITAL AREA BARRIER AS COMMITTED TO IN THE SECURITY PLAN. 2.790 INFORMATION.
(8004 INFRACTION)

AS REQUIRED BY 10 CFR 20.103(C) "WHEN RESPIRATORY PROTECTIVE EQUIPMENT IS USED TO LIMIT THE INHALATION OF AIRBORNE RADIOACTIVE MATERIAL PURSUANT TO PARAGRAPH (B)([illegible]) OF THIS SECTION, THE LICENSEE MAY MAKE ALLOWANCES FOR SUCH USE IN ESTIMATING EXPOSURES OF INDIVIDUALS TO SUCH MATERIALS PROVIDED THAT SUCH EQUIPMENT IS USED AS STIPULATED IN REGULATORY GUIDE 8.15, "ACCEPTABLE PROGRAMS FOR RESPIRATORY PROTECTION"." SECTION C.8.A OF REGULATORY GUIDE REQUIRES "RESPERIBLE AIR OF APPROVED QUALITY BE PROVIDED" IN ACCORDANCE WITH "COMMODITY SPECIFICATION FOR AIR", G--7.1-1966, WHICH IN SECTION 2.3, ENTITLED QUALITY TESTS FOR GRADE D AIR REQUIRES TEST FOR OXYGEN, HYDROCARBONS, CARBON MONOXIDE AND CARBON DIOXIDE. PROCEDURE HP-62, REVISION 2, DOES NOT PROVIDE FOR THE SAMPLING OR TESTING OF THE BREATHING AIR SUPPLY FOR OXYGEN, CONDENSED HYDROCARBONS OR CARBON DIOXIDE CONTENT AND THE SYSTEM WAS BEING USED WITHOUT THIS SAMPLING.
(8006 INFRACTION)

AS REQUIRED BY TECHNICAL SPECIFICATION 6.4.1, A RETRAINING AND REPLACEMENT TRAINING PROGRAM FOR THE FACILITY STAFF SHALL BE MAINTAINED UNDER THE DIRECTION OF THE TRAINING SUPERVISOR AND SHALL MEET OR EXCEED THE REQUIREMENTS AND RECOMMENDATIONS OF SECTIONS 5.5 OF ANSI N18.1-1971 AND APPENDIX "A" OF 10 CFR PART 55. SECTION 5.5 OF ANSI N18.1-1971 STATES THAT A TRAINING PROGRAM SHALL BE ESTABLISHED WHICH MAINTAINS THE PROFICIENCY OF THE OPERATING ORGANIZATION. THE LICENSEE DID NOT HAVE AN ADEQUATE RETRAINING PROGRAM WHICH MAINTAINED THE PROFICIENCY OF THE HEALTH PHYSICS STAFF, NOR DID THE LICENSEE HAVE A TRAINING PROGRAM FOR REPLACEMENT PERSONNEL ON THE HEALTH PHYSICS STAFF IN THAT THE BASIC ELEMENTS LISTED IN ANSI N18.1-1971 SECTION 5.5.1 WERE NOT CONTAINED IN THE PROGRAM.
(8006 INFRACTION)

AS REQUIRED BY TECHNICAL SPECIFICATION 6.9, PROCEDURES FOR PERSONNEL RADIATION PROTECTION SHALL BE PREPARED CONSISTENT WITH THE REQUIREMENTS OF 10 CFR PART 20 AND SHALL BE APPROVED, MAINTAINED AND ADHERED TO FOR ALL OPERATIONS INVOLVING PERSONNEL RADIATION EXPOSURE. PROCEDURE HP-30, REVISION 7, PARAGRAPH 8.3.6 REQUIRES THAT THE PLASTIC BAG HOLDING THE DOSIMETRY, THERMOLUMINESENT DOSIMETER (TLD) AND NEUTRON TLD BE TAPED TO THE FRONT OF THE UPPER TORSO AND PARAGRAPH 4.3 OF THE HEALTH PHYSICS MANUAL REQUIRES DOSIMETRY TO BE WORN ON THE FRONT SIDEOF THE UPPER THUNK. DURING THE PERIOD OF INSPECTION, NUMEROUS INSTANCES OF IMPROPER WEARING OF DOSIMETERS WERE OBSERVED. EXAMPLES INCLUDED TLD'S CLIPPED ON BELTS AT THE HIP SIDE, TLD'S SHIELDED BY POCKET DOSIMETER AND TLD'S TURNED BACKWARDS.
(8006 INFRACTION)

AS REQUIRED BY TECHNICAL SPECIFICATION 6.8.1 WRITTEN PROCEDURES SHALL BE ESTABLISHED, IMPLEMENTED AND MAINTAINED COVERING THE ACTIVITIES RECOMMENDED IN APPENDIX A OF REGULATORY GUIDE 1.33, NOVEMBER 1972. REGULATORY GUIDE 1.33, APPENDIX A, SECTION [illegible](C) STATES THAT WRITTEN PROCEDURES FOR SURVEYS AND MONITORING SHOULD BE PREPARED. PLANT PROCEDURE HP-41 STATES, IN PART, THAT IF AN ITEM IS TO BE REMOVED AS AN UNCONDITIONAL RELEASE FROM THE RADIATION CONTROL AREA, HEALTH PHYSICS SHALL ASCERTAIN THAT THE ITEM MEETS THE CRITERIA DESCRIBED IN PLANT PROCEDURE HP-71, PARAGRAPH 4.3.2. PROCEDURE HP-71, PARAGRAPH 4.3.2 STATES THAT THE RELEASE LIMIT FOR UNCONTROLLED MATERIAL LEAVING THE RCA IS 100 COUNTS PER MINUTE PER PROBE AREA (BETA-GAMMA) USING THE RM-14/HP-210 INSTRUMENT (APPROXIMATELY EQUIVALENT TO 1,000 DPM/PROBE AREA) AND NO DETECTABLE REMOVABLE CONTAMINATION. DURING THE INSPECTION A WRENCH WAS FOUND IN THE CLEAN TOOL ROOM LOCATED OUTSIDE THE RADIATION CONTROL AREA, WHICH WHEN TESTED WAS OBSERVED TO BE CONTAMINATED TO 50,000 DPM PER PROBE AREA.
(8006 INFRACTION)

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-018/ 03L-0	04/19/80	05/19/80	INOPERABLE DISCHARGE MONITOR.
80-020/ 03L-0	05/07/80	06/06/80	AUXFEED PUMP WOULD NOT DEVELOP SUFFICIENT DISCHARGE PRESSURE.
80-021/ 03L-0	05/06/80	06/06/80	D/G FAILED TO SYNCHRONIZE TO THE EMERGENCY BUS DURING POST MAINTENANCE TESTING.
80-024/ 03L-0	05/16/80	06/15/80	D/G LOAD SEQUENCERS FOUND TO BE OUT OF TOLERANCE DURING PERIODIC ESF TEST.
80-025/ 01T-0	05/12/80	05/26/80	INOPERABLE MSIV BYPASS VALVES.
80-027/ 09T-0	05/14/80	05/15/80	PART 21 REPORT. NSSS INFORMED LICENSEE THAT UNDER PARTIAL DRAIN OF PRIMARY (MODE 5) BORON DILUTION IS UNANALYZED.

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

NORMAL OPERATION.

* LAST IE SITE INSPECTION DATE: JUNE 17-20, 1980

* INSPECTION REPORT NO: 50-335/80-18

SUMMARY

INSPECTION SUMMARY

* INSPECTION MARCH 10-21 (80-06): THIS SPECIAL ANNOUNCED INSPECTION INVOLVED 487 INSPECTOR-HOURS ON SITE IN THE AREA OF HEALTH PHYSICS APPRAISAL INCLUDING ORGANIZATION, QUALIFICATIONS, TRAINING PROCEDURES, ALARM PROGRAMS, EXTERNAL EXPOSURE CONTROL, PERSONNEL DOSIMETER PROGRAM, INTERNAL DOSIMETRY, RESPIRATORY PROTECTION, MEDICAL EMERGENCIES, INSTRUMENTATION, SURVEILLANCE AND ACCESS CONTROL, RADWASTE CONTROL, FACILITIES AND EQUIPMENT, RE-ENTRY, AND IN-PLANT SYSTEMS. OF THE 16 AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN 12 AREAS; 4 ITEMS OF NONCOMPLIANCE WERE FOUND IN 4 AREAS (INFRACTION - FAILURE TO HAVE A RETRAINING PROGRAM; INFRACTION - FAILURE TO FOLLOW PROCEDURES; INFRACTION - FAILURE TO TAKE ADEQUATE SAMPLES OF BREATHING AIR; AND INFRACTION - CONTAMINATED TOOL CONTROL).

INSPECTION APRIL 10 - MAY 14 (80-15): THIS UNANNOUNCED INVESTIGATION INVOLVED 30 INSPECTOR-HOURS ON SITE IN THE AREAS OF MAINSTEAM AND ELECTRICAL PENETRATIONS; SAFETY INJECTION VALVES. NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION APRIL 29 - MAY 2 (80-12): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 21 INSPECTOR-HOURS ON SITE IN THE AREAS OF SURVEILLANCE ACTIVITY ON PIPE SUPPORT AND RESTRAINT SYSTEMS, INCLUDING EXAMINATION OF PROCEDURE TEST RESULTS AND INSTALLED RESTRAINTS, AND EXAMINATION OF RECORDS. OF THE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION JUNE 3-5 (80-16): ON MAY 22, 1980, ALLEGATIONS RELATIVE TO INADEQUATE PHYSICAL SECURITY AND POOR HEALTH PHYSICS CONTROL WERE TELEPHONICALLY RECEIVED BY THE U.S.N.R.C., ATLANTA REGIONAL OFFICE, FROM AN INDIVIDUAL WHO DESIRES CONFIDENTIALITY. ON JUNE 3, 1980,

INSPECTORS INTERVIEWED THIS INDIVIDUAL, INSPECTED THE FACILITY, EXAMINED DOCUMENTATION, AND CONDUCTED INTERVIEWS OF OTHER PLANT PERSONNEL, ENDING THE EFFORT ON JUNE 5, 1980. THE FIVE SPECIFIC ALLEGATIONS WERE UNSUBSTANTIATED. THERE WERE NO ITEMS OF NONCOMPLIANCE.

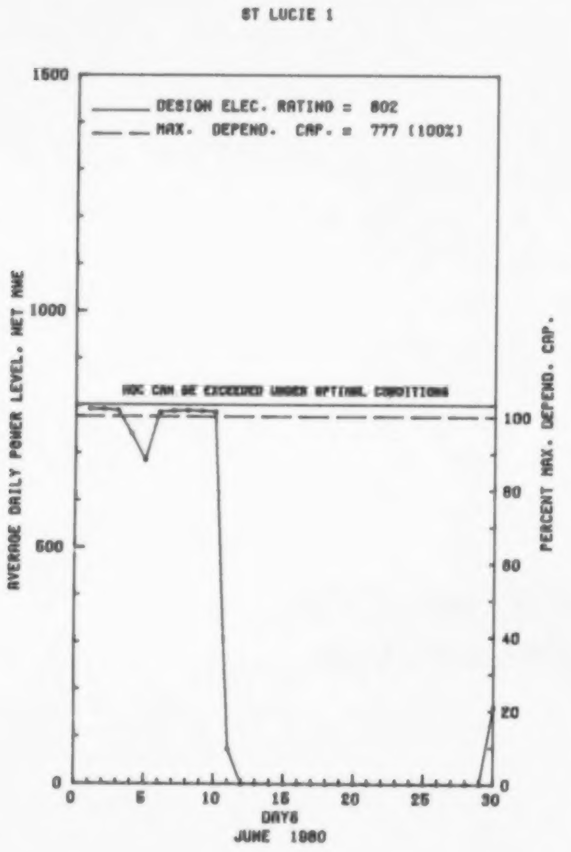
INSPECTION JUNE 9-10 (80-19): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED TEN INSPECTOR-HOURS ON SITE IN THE AREAS OF FOLLOWUP ON IE BULLETINS AND FOLLOWUP ON PREVIOUS INSPECTION FINDINGS. OF THE TWO AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

REVIEWED BY OMPA LOEB: MR Beebe **DATE:** 08/04/80

REVIEWED BY NRR DL: **DATE:**

ST. LUCIE 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-335
2. Utility Contact: V. T. Chilson (305) 552-3824
3. Licensed Thermal Power (MWt): 2560
4. Nameplate Rating (Gross MWe): 1000 x 0.85 = 850
5. Design Electrical Rating (Net MWe): 802
6. Maximum Dependable Capacity (Gross MWe): 822
7. Maximum Dependable Capacity (Net MWe): 777
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	4,367.0	30,911.0
12. Hours Reactor Was Critical	256.0	2,536.2	24,046.1
13. Reactor Reserve Shutdown Hours	.0	.0	129.4
14. Hours Generator On-Line	252.2	2,444.4	23,274.5
15. Unit Reserve Shutdown Hours	.0	.0	32.0
16. Gross Thermal Energy (MWH)	625,[illegible]13	6,003,127	56,262,693
17. Gross Electrical Energy (MWH)	203,250	1,937,780	18,232,280
18. Net Electrical Energy (MWH)	188,166	1,816,304	17,144,200
19. Unit Service Factor	35.0	56.0	75.3
20. Unit Availability Factor	35.0	56.0	75.4
21. Unit Capacity Factor (MDC Net)	33.6	53.5	71.4
22. Unit Capacity Factor (DER Net)	32.6	51.9	69.2
23. Unit Forced Outage Rate	65.0	16.5	6.7

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
8	06/04/80	S	.8	B	5		HH	FILTER	LOAD REDUCTION TO APPROXIMATELY 50% R.P. TO CLEAN STEAM GENERATOR FEEDWATER PUMP NO. 1B SUCTION STRAINER. (NON-NUCLEAR SYSTEM).
9	06/11/80	F	467.8	A	2		WB	VALVOP	REACTOR WAS MANUALLY TRIPPED WHEN COMPONENT COOLING WATER TO REACTOR COOLANT PUMP MECHANICAL SEALS WAS LOST AND COULD NOT BE IMMEDIATELY RESTORED DUE TO CLOSURE OF HCV-14-6. CORRECTIVE ACTIONS INCLUDED PROVIDING A BACKUP NITROGEN SUPPLY TO THE DIAPHRAGM OPERATED COMPONENT COOLING WATER VALVES. ELECTRICAL CONDUIT WAS REPAIRED TO PREVENT ENTRY OF WATER. OUTAGE WAS EXTENDED TO INSPECT MECHANICAL SEALS ON REACTOR COOLANT PUMPS.

SUMMARY

THE UNIT OPERATED AT 100% POWER PRODUCTION FROM THE BEGINNING OF THE MONTH UNTIL 06/11/80 WITH A BRIEF REDUCTION ON 06/09/80 FOR CLEANING STEAM GENERATOR PUMP. ON 06/11/80, THE REACTOR WAS MANUALLY SCRAMMED TO INSPECT COOLANT PUMPS.

LEGEND

[1]Type:
F--Forced
S--Scheduled

[2]Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3]Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4]Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5]Exhibit H:
Same source.

SURRY 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Virginia/Surry 17 miles NW of Newport News
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2441
5.	Design Electrical Rating (MWE Net)	822
6.	Date of Initial Criticality	July 1, 1972
7.	Date of Commercial Operation	December 22, 1972
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	James River

Utility & Contractor Information

10.	Licensee	Virginia Electric and Power Company
11.	Corporate Address	P. O. Box 26666 Richmond, Virginia 23261
12.	Corporate Contact	J. H. Ferguson Exec. V.P. - Power
13.	Architect/Engineer	Stone & Webster
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R.C. Lewis
18.	IE Section Chief	P. J. Kellogg
19.	IE Resident Inspector	D. Burke (804) 357-2101
20.	Licensing Project Manager	D. Neighbors
21.	Docket Number	50-280
22.	License Number/Date of Issue	DPR-32, 5/25/72
23.	Public Document Room	Swem Library College of William and Mary Williamsburg, Virginia 23185

INSPECTION STATUS

ENFORCEMENT STATUS

10CFR71.5 REQUIRES DOT REGULATIONS BE ADHERED TO FOR RADIOACTIVE WASTE SHIPMENTS. 49CFR173.393(N)(9) REQUIRES THE SHIPPER TO INSURE BY EXAMINATION OR APPROPRIATE TEST THAT EXTERNAL RADIATION LEVELS ARE WITHIN ALLOWABLE LIMITS. SURVEYS CONDUCTED ON A VEHICLE CARRYING RADIOACTIVE WASTE ON APRIL 14, 1980 WERE INADEQUATE IN THAT THEY FAILED TO IDENTIFY SURFACE RADIATION LEVELS IN EXCESS OF REGULATORY LIMITS. (8016 INFRACTION)

10CFR71.5 REQUIRES DOT REGULATION BE ADHERED TO FOR RADIOACTIVE WASTE SHIPMENTS. 49CFR173.393(I)(2) REQUIRES RADIATION LEVFLS ON THE SURFACE OF A SOLE USE VEHICLE NOT EXCEED 200 MREM/HOUR. ON APRIL 14, 1980 A SHIPMENT LEFT THE SITE WITH A SURFACE RADIATION LEVEL AS HIGH AS 270 TO 300 MREM/HOUR.
(8016 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

UNIT IS CURRENTLY OPERATING AT 90% POWER.

LAST IE SITE INSPECTION DATE: JUNE 6, 1980

INSPECTION REPORT NO: 50-280/80-20

SUMMARY

INSPECTION SUMMARY

INSPECTION MAY 27-30 (80-17): INCLUDED REVIEW OF SECURITY PROGRAM AUDIT; SECURITY ORGANIZATION; PROTECTED AND VITAL AREAS BARRIERS; ACCESS CONTROL (IDENTIFICATION, BADGING AND AUTHORIZATION); ACCESS CONTROL (VITAL AREA); ACCESS CONTROL (ESCORTING) AND COMMUNICATION CONTROLS. THE INSPECTION INVOLVED 11 INSPECTOR-HOURS ON SITE BY ONE NRC INSPECTOR. THE INSPECTION WAS BEGUN DURING AN OFFSHIFT PERIOD. TWO INSPECTION HOURS WERE ACCOMPLISHED DURING OFFSHIFT PERIODS. OF THE AREAS INSPECTED, NO APPARENT ITEMS OF NON-COMPLIANCE WERE DISCLOSED.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

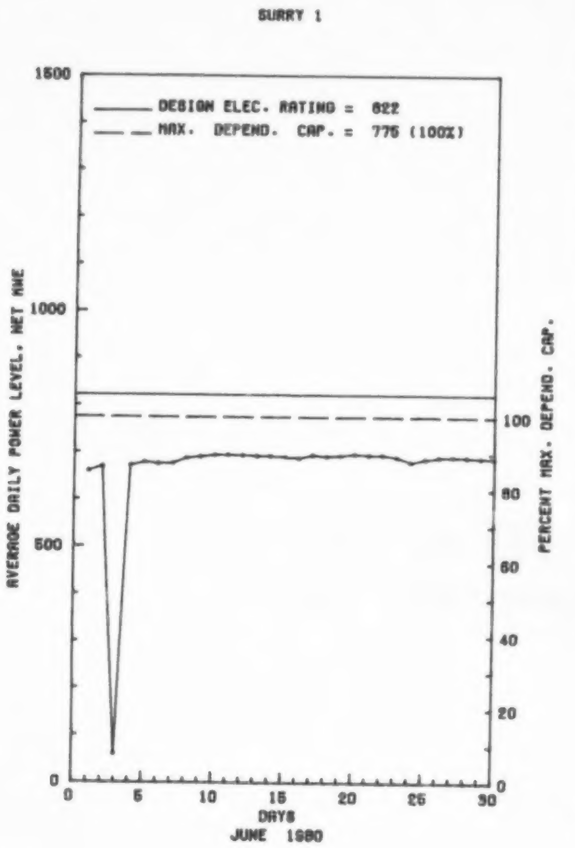
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE			

REVIEWED BY OMPA LOEB: Earl Hill **DATE:** 8-4-80

REVIEWED BY NRR DL: Don Neighbors **DATE:** 8/5/80

SURRY 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-280
2. Utility Contact: SUE DUNN (804) 357-3184
3. Licensed Thermal Power (MWt): 2441
4. Nameplate Rating (Gross MWe): 942 x 0.9 = 848
5. Design Electrical Rating (Net MWe): 822
6. Maximum Dependable Capacity (Gross MWe): 811
7. Maximum Dependable Capacity (Net MWe): 775
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): 784
10. Reasons for Restrictions, If Any: TURBINE BLADING PROBLEMS

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	4,367.0	65,951.0
12. Hours Reactor Was Critical	705.7	2,209.7	40,961.3
13. Reactor Reserve Shutdown Hours	.0	182.6	3,731.5
14. Hours Generator On-Line	701.5	2,193.1	40,098.1
15. Unit Reserve Shutdown Hours	.0	183.6	3,736.2
16. Gross Thermal Energy (MWH)	1,699,785	5,268,805	92,983,677
17. Gross Electrical Energy (MWH)	507,900	1,631,460	30,311,958
18. Net Electrical Energy (MWH)	479,886	1,545,886	28,772,785
19. Unit Service Factor	97.4	50.2	60.8
20. Unit Availability Factor	97.4	54.4	66.5
21. Unit Capacity Factor (MDC Net)	86.0	45.7	56.3
22. Unit Capacity Factor (DER Net)	81.1	43.1	53.1
23. Unit Forced Outage Rate	2.6	49.8	26.8

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): 09/15/80 - S/G REPLACEMENT. (1 YEAR)
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-10	06/03/80	F	18.5	A	3	80-29/03L-0			EXPERIENCED A TURBINE RUNBACK, REACTOR TRIP-TURBINE TRIP AND SAFETY INJECTION INITIATION UPON LOSS OF POWER TO THE I-IV VITAL BUS. THE LOSS OF THE I-IV VITAL BUS WAS CAUSED BY AN ELECTRICAL FIRE IN THE SOLA TRANSFORMER SUPPLYING POWER TO IT.

SUMMARY

THE REACTOR RAN NORMALLY FOR THE MONTH WITH AN OUTAGE ON 06/03/80 FOR A SCRAM DUE TO AN ELECTRICAL FIRE IN THE POWER SUPPLY OF THE TURBINE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

SURRY 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Virginia/Surry 17 miles NW of Newport News
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2441
5.	Design Electrical Rating (MWE Net)	822
6.	Date of Initial Criticality	March 7, 1973
7.	Date of Commercial Operation	May 1, 1973
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	James River

Utility & Contractor Information

10.	Licensee	Virginia Electric and Power Company
11.	Corporate Address	P. O. Box 26666 Richmond, Virginia 23261
12.	Corporate Contact	J. H. Ferguson Exec. V.P. - Power
13.	Architect/Engineer	Stone & Webster
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R.C. Lewis
18.	IE Section Chief	P. J. Kellog
19.	IE Resident Inspector	D. Burke (804) 357-2101
20.	Licensing Project Manager	D. Neighbors
21.	Docket Number	50-281
22.	License Number/Date of Issue	DPR-37, 1/29/73
23.	Public Document Room	Swem Library College of William and Mary Williamsburg, Virginia 23185

INSPECTION STATUS

ENFORCEMENT STATUS

10CFR71.5 REQUIRES DOT REGULATIONS BE ADHERED TO FOR RADIOACTIVE WASTE SHIPMENTS. 49CFR173.393(N)(9) REQUIRES THE SHIPPER TO INSURE BY EXAMINATION OR APPROPRIATE TEST THAT EXTERNAL RADIATION LEVELS ARE WITHIN ALLOWABLE LIMITS. SURVEYS CONDUCTED ON A VEHICLE CARRYING RADIOACTIVE WASTE ON APRIL 14, 1980 WERE INADEQUATE IN THAT THEY FAILED TO IDENTIFY SURFACE RADIATION LEVELS IN EXCESS OF REGULATORY LIMITS.
(8017 INFRACTION)

10CFR71.5 REQUIRES DOT REGULATION BE ADHERED TO FOR RADIOACTIVE WASTE SHIPMENTS. 49CFR173.393(I)(2) REQUIRES RADIATION LEVELS ON THE SURFACE OF A SOLE USE VEHICLE NOT EXCEED 200 MREM/HOUR. ON APRIL 14, 1980 A SHIPMENT LEFT THE SITE WITH A SURFACE RADIATION LEVEL AS HIGH AS 270 TO 300 MREM/HOUR.
(8017 INFRACTION)

AS REQUIRED BY TECHNICAL SPECIFICATIONS 6.4.A.7 AND 6.4.D, DETAILED WRITTEN PROCEDURES WITH APPROPRIATE CHECK-OFF LISTS AND INSTRUCTIONS SHALL BE PROVIDED AND FOLLOWED FOR PREVENTIVE OR CORRECTIVE MAINTENANCE OPERATIONS WHICH WOULD HAVE AN EFFECT ON THE SAFETY OF THE REACTOR. CONTRARY TO THE ABOVE, THE "A" REACTOR COOLANT LOOP HYDROSTATIC TEST PROCEDURE (DC 50006-P-4-U2A) PERFORMED ON JANUARY 30, 1980 DID NOT: (A) SPECIFY THE POSITION OF ALL BOUNDARY VALVES ASSOCIATED WITH THE TEST; (B) INCORPORATE FIELD CHANGES DISCOVERED DURING THE "B" AND "C" HYDROSTATIC TESTS THOUGH THE ACTION SPECIFIED BY THE FIELD CHANGES WAS TAKEN; (C) SPECIFICALLY IDENTIFY REMOVAL OF ALL PROCEDURE INSTALLED TEST EQUIPMENT; (D) SATISFACTORILY REVIEW THAT THE ALPHA-NUMERIC VALVE DESIGNATORS APPLIED TO THE VALVE DESCRIPTIONS.
(8020 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

SYSTEM MODIFICATIONS AND REANALYSIS FOR SEISMIC SUPPORTS ARE CONTINUING.

FACILITY ITEMS (PLANS AND PROCEDURES):

ESTIMATE PLANT STARTUP JULY 7, 1980.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

UNIT IS CURRENTLY SHUT DOWN CONDUCTING PREOPERATIONAL TESTING. THE UNIT HAS BEEN REFUELED.

LAST IE SITE INSPECTION DATE: JUNE 6, 1980

INSPECTION REPORT NO: 50-281/80-21

SUMMARY

INSPECTION SUMMARY

INSPECTION MAY 27-30 (80-23): INCLUDED REVIEW OF SECURITY PROGRAM AUDIT; SECURITY ORGANIZATION; PROTECTED AND VITAL AREAS BARRIERS; ACCESS CONTROL (IDENTIFICATION, BADGING AND AUTHORIZATION); ACCESS CONTROL (VITAL AREA); ACCESS CONTROL (ESCORTING) AND COMMUNICATION CONTROLS. THE INSPECTION INVOLVED 11 INSPECTOR-HOURS ON SITE BY ONE NRC INSPECTOR. THE INSPECTION WAS BEGUN DURING AN OFFSHIFT PERIOD; TWO INSPECTION HOURS WERE ACCOMPLISHED DURING OFFSHIFT PERIODS. OF THE AREAS INSPECTED, NO APPARENT ITEMS OF NON-COMPLIANCE WERE DISCLOSED IN SEVEN AREAS; ONE APPARENT ITEM OF NONCOMPLIANCE WAS IDENTIFIED IN ONE AREA (ACCESS CONTROL - FAILURE TO MAINTAIN POSITIVE CONTROL OF PERSONNEL DURING MAJOR OUTAGE).

REPORTS RECEIVED FROM LICENSEE

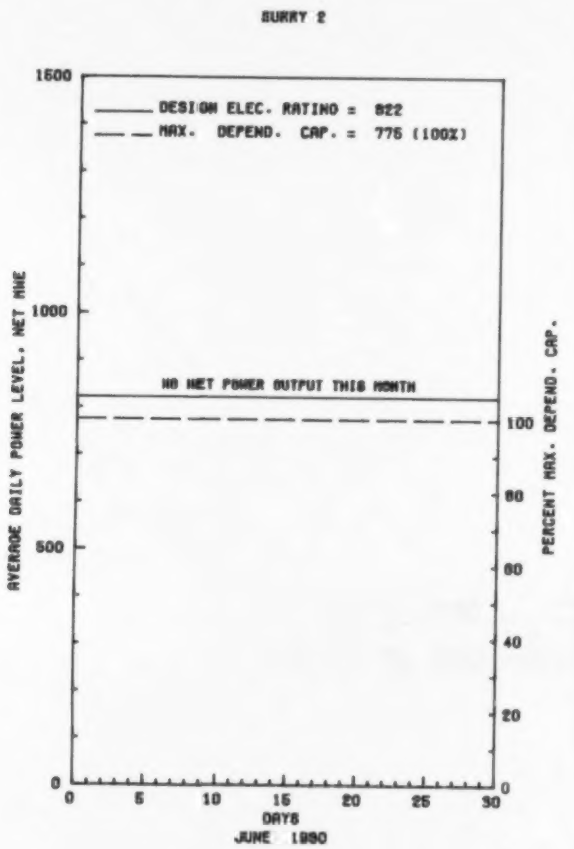
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
NONE.				

REVIEWED BY OMPA LOEB: DATE: 8 4 80

REVIEWED BY NRR DL: Don Neighbors DATE: 8/5/80

SURRY 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-281
2. Utility Contact: SUE DUNN (804) 357-3184
3. Licensed Thermal Power (MWt): 2441
4. Nameplate Rating (Gross MWe): 942 x 0.9 = 848
5. Design Electrical Rating (Net MWe): 822
6. Maximum Dependable Capacity (Gross MWe): 811
7. Maximum Dependable Capacity (Net MWe): 775
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	4,367.0	62,831.0
12. Hours Reactor Was Critical	.0	.0	34,499.9
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	.0	.0	33,996.2
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	0	79,194,083
17. Gross Electrical Energy (MWH)	0	0	25,868,844
18. Net Electrical Energy (MWH)	0	0	24,536,605
19. Unit Service Factor	.0	.0	54.1
20. Unit Availability Factor	.0	.0	54.1
21. Unit Capacity Factor (MDC Net)	.0	.0	50.4
22. Unit Capacity Factor (DER Net)	.0	.0	47.5
23. Unit Forced Outage Rate	.0	.0	21.0

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 07/10/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-6	02/04/79	S	720.0	D	4		RC	FUELXX	CONTINUATION OF SHUTDOWN FOR REFUELING AND STEAM GENERATOR REPLACEMENT WHICH BEGAN ON 02/04/79. WORK STILL UNDERWAY ON PIPING RESTRAINT MODIFICATIONS REQUIRED PRIOR TO UNIT RECOVERY.

SUMMARY

REFUELING AND MAINTENANCE OUTAGE CONTINUING.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

THREE MILE ISLAND 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1. Location (state/county) (nearest population center)	Pennsylvania/Dauphin 10 miles SE of Harrisburg
2. Electric Reliability Council	Mid-Atlantic Area Council
3. Type of Reactor	PWR
4. Power Level (MWT)	2535
5. Design Electrical Rating (MWE Net)	819
6. Date of Initial Criticality	June 5, 1974
7. Date of Commercial Operation	September 2, 1974
8. Condenser Cooling Method	Closed
9. Condenser Cooling Water Source	Susquehanna River

Utility & Contractor Information

10. Licensee	Metropolitan Edison Company
11. Corporate Address	P. O. Box 542 Reading, Pennsylvania
12. Corporate Contact	R. C. Arnold, Sr. Vice-President 260 Cherry Hill Rd. Parsippany, NJ 07054
13. Architect/Engineer	Gilbert Associates, Inc.
14. Nuclear Steam Supply System	B&W
15. Constructor	UE&C

Regulatory Information

16. IE Region responsible	I
17. IE Branch Chief	E. J. Brunner
18. IE Section Chief	A. N. Fasano
19. NRC-TMI Support Staff	D. Haverkamp FTS: 590-3950 (717) 782-3950
20. Licensing Project Manager	D. C. DiIanni / H. Silver (X27435)
21. Docket Number	50-289
22. License Number/Date of Issue	DPR-50, 4/19/74
23. Public Document Room	Government Publications Section State Library of Pennsylvania Education Building Commonwealth and Walnut Street Harrisburg, Pennsylvania 17126

INSPECTION STATUS

ENFORCEMENT STATUS

THE REQUIREMENTS OF RRR (RECOMMENDED REQUIREMENTS FOR THE RESTART OF TMI, UNIT 1) IMPOSTED BY NRC-NRR IN ADDITION TO THE LICENSEES QA PROGRAM, IS NOT BEING INCORPORATED OR REFERENCED IN THE LICENSEES IMPLEMENTING INSTRUCTIONS/PROCEDURES INCLUDING ENGINEERING TASKS ASSIGNED THE A-E.
(8005 DEVIATION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS:

The plant remains shutdown, by NRC Order, pending completion of modifications and other actions related to the TMI-2 accident.

+ LAST IE SITE INSPECTION DATE: 6/27/80

+ INSPECTION REPORT NO: 50-289/80-17

SUMMARY

INSPECTION SUMMARY

+ 50-289/80-05 - Feb 27-29: Special, announced inspection by two regional based inspectors (49 hours) of the Quality Assurance Program implementation in the areas of Design Changes/Modifications; Procurement; and, Unit 1 Restart Program. A deviation was identified: Failure to incorporate recommended restart requirements into established Engineering Department procedures.

+ 50-289/80-09 - Apr 1 - May 8: Routine, unannounced inspection by six regional based inspectors (188 hours) of work activities and records associated with the Unit 1 Restart Program, including: inservice inspection observations; ISI evaluation and documentation; pressurizer heaters emergency power supply (task RM-16); tendon surveillance program; hydrogen recombiner (task RM-12); fire protection (task NM-48); engineering design; and, identification of safety-related ECMs. No items of noncompliance were identified.

+ 50-289/80-11 - Apr 28 - May 2: Special, announced inspection by one regional based inspector (35 hours) of the Quality Assurance Program implementation in the areas of Design Changes/Modifications; Audits; QA/QC Surveillance; and the Unit 1 Restart Program. No items of noncompliance were identified.

REPORTS RECEIVED FROM LICENSEE

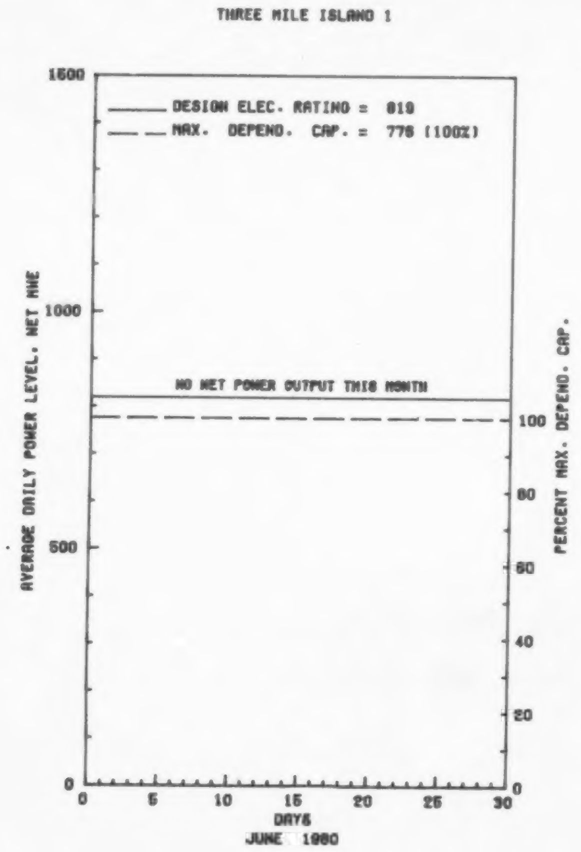
DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT

NONE

REVIEWED BY OMPA LOEB: PE Robe **DATE:** 8/4/80
REVIEWED BY NRR DL: D C Dilanni **DATE:** 8/5/80

THREE MILE ISLAND 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-289
2. Utility Contact: D. G. Mitchell (215) 921-6579
3. Licensed Thermal Power (MWt): 2535
4. Nameplate Rating (Gross MWe): 968 x 0.9 = 871
5. Design Electrical Rating (Net MWe): 819
6. Maximum Dependable Capacity (Gross MWe): 840
7. Maximum Dependable Capacity (Net MWe): 776
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	4,367.0	51,096.0
12. Hours Reactor Was Critical	.0	.0	31,731.8
13. Reactor Reserve Shutdown Hours	.0	.0	838.5
14. Hours Generator On-Line	.0	.0	31,180.9
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	0	76,531,071
17. Gross Electrical Energy (MWH)	0	0	25,484,330
18. Net Electrical Energy (MWH)	0	0	23,840,053
19. Unit Service Factor	.0	.0	61.0
20. Unit Availability Factor	.0	.0	61.0
21. Unit Capacity Factor (MDC Net)	.0	.0	60.1
22. Unit Capacity Factor (DER Net)	.0	.0	57.0
23. Unit Forced Outage Rate	100.0	100.0	28.6

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: NO INFORMATION

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
1	02/17/79	F	720.0	D	4				REGULATORY RESTRAINT ORDER.

SUMMARY

THE UNIT REMAINS SHUTDOWN BY THE NRC FOLLOWING THE ACCIDENT AT TMI-2.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

THREE MILE ISLAND 2

NRC POWER RESTRICTIONS: SEE "LICENSE STATUS"

FACILITY DATA

Facility Description

1. Location (state/county) (nearest population center)	Pennsylvania/Dauphin 10 miles SE of Harrisburg
2. Electric Reliability Council	Mid-Atlantic Area Council
3. Type of Reactor	PWR
4. Licensee Power Level (MWT)	2772
5. Design Electrical Rating (MWE Net)	906
6. Date of Initial Criticality	3/28/78
7. Date of Commercial Operation	12/30/78
8. Condenser Cooling Method	CLOSED
9. Condenser Cooling Water Source	Susquehanna River

Utility & Contractor Information

10. Licensee	Metropolitan Edison Company
11. Corporate Address	P. O. Box 542 Reading, PA 19603
12. Corporate Contact	R. C. Arnold, Sr. Vice-President 100 Interpace Parkway Parsippany, NJ 07054
13. Architect/Engineer	Burns & Roe
14. Nuclear Steam Supply System	B&W
15. Constructor	UE&C

Regulatory Information

16. IE Region responsible	I
17. IE Branch Chief	E. J. Brunner
18. IE Section Chief	A. N. Fasano (717) 782-3950 FTS: 590-3950
19. IE Resident Inspector	R. Conte (717) 782-3950 FTS: 590-3950
20. TMI Program Director	B. Snyder
21. Docket Number	50-320
22. License Number/Date of Issue	DPR-73/2; 2/8/78
23. Public Document Room	Government Publications Section State Library of Pennsylvania Education Building Commonwealth & Walnut Streets Harrisburg, Pennsylvania 17126

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEM AND COMPONENT PROBLEMS:

- Cleanup operations from 3/28/79 accident.

PLANT STATUS:

- Decay heat is being removed by cyclic natural circulation through the "A" Once Through Steam Generator and to the reactor building ambient from both RCS loops.
- LAST IE SITE INSPECTION DATE: 6/30/80
- INSPECTION REPORT NO: 50-320/80-11

SUMMARY

INSPECTION SUMMARY

- None.

LICENSE STATUS

By Order for Modification of License dated July 20, 1979, the licensee's authority to operate the facility was suspended and the licensee's authority was limited to maintenance of the facility in the present shutdown cooling mode (44 F.R. 45271).

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-016/01L	04/22/80	05/23/80	Boric acid mix tank boron concentration greater than Technical Specification limit
80-019/01L	05/18/80	06/17/80	Inoperable B steam generator level transmitter

REVIEWED BY OMPA LOEB: P E Rote **DATE:** 8/4/80

REVIEWED BY NRR TMIPO: D. J. Brinkman **DATE:** 8/5/80

THREE MILE ISLAND 2

AVERAGE DAILY POWER LEVEL (MWe)

OPERATING STATUS

1. Docket Number: 50-320
2. Utility Data Prepared By: R. A. Lengel (215) 921-6681
3. Licensed Thermal Power (MWt): 2772
4. Nameplate Rating (Gross MWe): 1068 X 0.9 = 961
5. Design Electrical Rating (Net MWe): 906
6. Maximum Dependable Capacity (Gross MWe): 0
7. Maximum Dependable Capacity (Net MWe): 0
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons
None
9. Power Level To Which Restricted, If Any (Net MWe): None
10. Reasons For Restrictions, If Any:

This Month Yr

11. Hours In Reporting Period
12. Number Of Hours Reactor Was Critical
13. Reactor Reserve Shutdown Hours
14. Hours Generator On-Line
15. Unit Reserve Sh
16. Gro
(MWH)
19
20. U Availability Factor
21. Unit Capacity Factor (Using MDC Net) 1/
22. Unit Capacity Factor (Using DER Net) 1/
23. Unit Forced Outage Rate
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
None
25. If Shut Down At End Of Report Period, Estimated Date of Startup: N/A

NO DECISION ON FUTURE OPERATION
SHUTDOWN INDEFINITELY

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
SEE SUMMARY BELOW.									

SUMMARY

THE UNIT HAS REMAINED SHUTDOWN SINCE THE 03/28/79 ACCIDENT, WHICH CAUSED EXTENSIVE CORE DAMAGE.

AUTHORITY TO OPERATE FACILITY SUSPENDED BY NRC ORDER DATED JULY 20, 1979, WHICH REQUIRES LICENSEE TO MAINTAIN FACILITY IN A SHUTDOWN CONDITION.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

TROJAN

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Oregon/Columbia 42 miles N of Portland
2.	Electric Reliability Council	Western Systems Coordinating Council
3.	Type of Reactor	PWR
4.	License Power Level (MWT)	3411
5.	Design Electrical Rating (MWE Net)	1130
6.	Date of Initial Criticality	December 15, 1975
7.	Date of Commercial Operation	May 20, 1976
8.	Condenser Cooling Method	Natural draft cooling tower
9.	Condenser Cooling Water Source	Columbia River

Utility & Contractor Information

10.	Licensee	Portland General Electric Co.
11.	Corporate Address	121 S. W. Salmon St. Portland, Oregon 97204
12.	Corporate Contact	C. Goodwin, Jr., Asst. V.P.
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Multiple

Regulatory Information

16.	IE Region responsible	V
17.	IE Branch Chief	J. L. Crews
18.	IE Section Chief	D. M. Sternberg
19.	IE Resident Inspector	M. H. Malmros (503) 556-3751
20.	Licensing Project Manager	C. M. Trammell
21.	Docket Number	50-344
22.	License Number/Date of Issue	NPF-1, 11/21/75
23.	Public Document Room	Columbia County Courthouse Law Library Circuit Court Room St. Helens, Oregon 97501

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

SYSTEMS AND COMPONENT PROBLEMS:

Resolution of questions regarding concrete block walls is required prior to startup.

FACILITY ITEMS (PLANS AND PROCEDURES):

None.

MANAGERIAL ITEMS:

None.

PLANT STATUS

♦ The plant commenced a refueling and maintenance outage on April 11, 1980. Startup is scheduled for mid-July 1980.

LAST IE SITE INSPECTION DATE: 06/01-30/80+

INSPECTION REPORT NO: 50-344/80-14+

SUMMARY

INSPECTION SUMMARY:

♦ Inspection on April 8-11, 16-18 & May 6-9, 1980 (Report No. 50-344/80-08)
Areas Inspected: Routine, announced inspection by Regional based inspectors of inservice inspection activities; maintenance activities related to pressurizer code safety valves; preparations for the control building wall modifications; and activities performed in response to IE Bulletin 79-01B. The inspection involved 114 inspector hours onsite by four NRC inspectors.

Results: No items of noncompliance or deviations were identified.

♦ Inspection on May 1-30, 1980 (Report No. 50-344/80-11)
Areas Inspected: Routine inspections of plant operations; maintenance; surveillance testing; physical security; refueling; modifications; environmental protection; and followup on Licensee Event Reports. The inspection involved 177 inspector hours onsite by the NRC Resident Inspector.

Results: Two items of noncompliance were found (Infraction - Failure to properly review and approve a safety-related test procedure; Deficiency - No record of certification available for individuals performing safety-related tests. No deviations were found.

♦ Inspection on May 14-16, 1980 (Report No. 50-344/80-12)
Areas Inspected: Routine, unannounced inspection of response to IE Bulletins and Circulars and followup on previous inspection items. Inspection also included independent inspection effort. The inspection involved 15 inspector hours onsite by one NRC inspector.

Results: No items of noncompliance or deviations were identified.

♦ Inspection on June 10-11, 1980 (Report No. 50-344/80-13)
Areas Inspected: Special announced inspection of the circumstances surrounding and corrective actions taken by the licensee as a consequence of Licensee Event Report 80-07 and selected corrective actions taken by the licensee as a consequence of recent findings related to Licensee Event Report 79-15. The inspection involved 24 inspector hours onsite by one NRC inspector.

Results: No items of noncompliance or deviations were identified.

♦ Inspection on June 1-30, 1980 (Report No. 50-344/80-14)
Report being prepared; to be reported next month.

REPORTS RECEIVED FROM LICENSEE

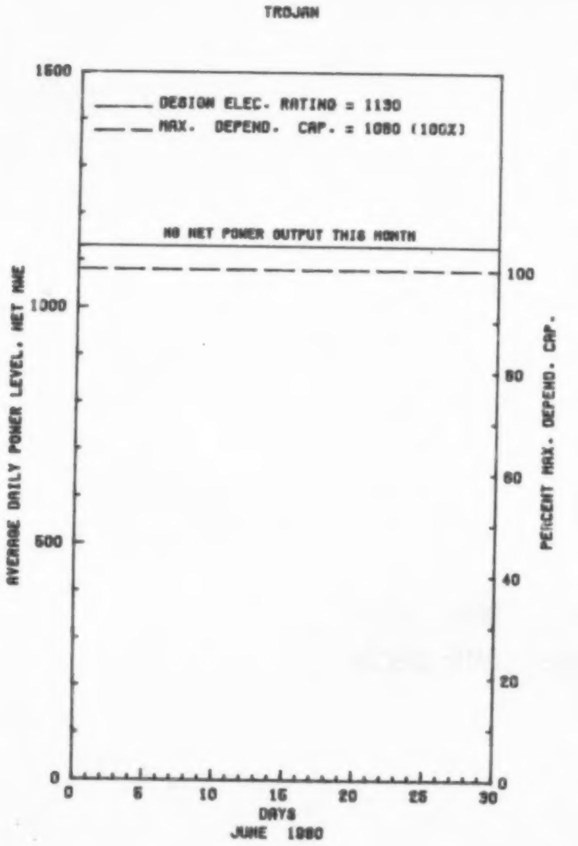
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-07/01T-0	05-16-80	05-30-80	Single-wythe block wall in auxiliary building not properly connected at top of wall.
80-08/01T-0	05-16-80	05-30-80	Containment integrity not maintained while moving fuel.
80-09/03L-0	05-22-80	06-20-80	Inadvertent partial draining of spent fuel pool.

REVIEWED BY OMPA LOEB: P.E. Bobe **DATE:** 8/4/80

REVIEWED BY NRR DL: P.C. Wagner **DATE:** 8/5/80

TROJAN

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-344
2. Utility Contact: G. G. Bair (503) 556-3713 x234
3. Licensed Thermal Power (MWt): 3411
4. Nameplate Rating (Gross MWe): 1280 x 0.95 = 1216
5. Design Electrical Rating (Net MWe): 1130
6. Maximum Dependable Capacity (Gross MWe): 1122
7. Maximum Dependable Capacity (Net MWe): 1080
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe):
10. Reasons for Restrictions, If Any: NRC LICENSING QUESTIONS ABOUT SEISMIC INTEGRITY AUX BLDG.

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	4,367.0	33,575.0
12. Hours Reactor Was Critical	.0	2,422.0	18,720.0
13. Reactor Reserve Shutdown Hours	.0	.0	2,171.0
14. Hours Generator On-Line	.0	2,416.0	18,044.6
15. Unit Reserve Shutdown Hours	.0	.0	1,508.7
16. Gross Thermal Energy (MWH)	0	6,839,434	55,735,044
17. Gross Electrical Energy (MWH)	0	2,203,790	18,189,205
18. Net Electrical Energy (MWH)	-3,638	2,088,895	17,122,214
19. Unit Service Factor	.0	55.3	53.7
20. Unit Availability Factor	.0	55.3	58.2
21. Unit Capacity Factor (MDC Net)	.0	44.3	47.2
22. Unit Capacity Factor (DER Net)	.0	42.3	45.1
23. Unit Forced Outage Rate	100.0	8.2	28.3

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): DESIGN MODIFICATIONS FROM TMI, OCTOBER 1980, 30 DAYS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: 07/12/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-04	04/11/80	S	528.0	C	4		RC	FUELXX	COMPLETION OF REFUELING OUTAGE (06/10/80).
	06/22/80	F	192.0	H	9	LER-80-07	ZZ	ZZZZZZ	AUXILIARY BUILDING SOUTH-WALL DISCOVERED NOT TO BE CONNECTED AT TOP OF WALL TO INTERFACING STRUCTURES.

SUMMARY

THE UNIT BEGAN THE MONTH IN A REFUELING OUTAGE WHICH WAS COMPLETED ON 06/10/80. THE OUTAGE WAS CONTINUED FOR THE REST OF THE MONTH FOR MODIFICATIONS TO AUXILIARY BUILDING WALLS.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

TURKEY POINT 3

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Florida/Dade 25 miles S of Miami
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2200
5.	Design Electrical Rating (MWE Net)	693
6.	Date of Initial Criticality	October 20, 1972
7.	Date of Commercial Operation	December 14, 1972
8.	Condenser Cooling Method	Recirculating Canal System
9.	Condenser Cooling Water Source	Atlantic Ocean

Utility & Contractor Information

10.	Licensee	Florida Power and Light Company
11.	Corporate Address	9250 West Flagler Street P. O. Box 013100 Miami, Florida 33101
12.	Corporate Contact	R. E. Uhrig, V.P. Nuclear Affairs
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. Lewis
18.	IE Section Chief	R. D. Martin
19.	IE Resident Inspector	R. J. Vogt-Lowell (305) 245-7669
20.	Licensing Project Manager	M. Grotenhuis (301) 492-7128
21.	Docket Number	50-250
22.	License Number/Date of Issue	DPR-31, 7/19/72
23.	Public Document Room	Environmental and Urban Affairs Library Florida International University Miami, Florida 33199

INSPECTION STATUS

ENFORCEMENT STATUS

AS REQUIRED BY 10 CFR 50, APPENDIX B, CRITERION XVI, AND TOPICAL QUALITY REQUIREMENT 16.0 OF THE TOPICAL QUALITY ASSURANCE REPORT NO. FPLTQAR 1-76, AS IMPLEMENTED IN PART BY PARAGRAPH 5.2.2.3 OF QUALITY PROCEDURE 16.1, CORRECTIVE ACTION FOR ITEMS OF NONCOMPLIANCE IDENTIFIED IN NRC INSPECTION REPORTS SHALL BE TRACKED BY THE PLANT. CONTRARY TO THE ABOVE, THE CORRECTIVE ACTION DESCRIBED IN YOUR DECEMBER 18, 1979 RESPONSE TO THE ITEM OF NONCOMPLIANCE DESCRIBED IN PARAGRAPH 7 OF REPORT 50-250/251/79-31 WAS NOT TRACKED BY THE PLANT THEREBY CONTRIBUTING TO THE LACK OF IMPLEMENTATION OF THE CORRECTIVE ACTION BY YOUR STATED DUE DATE.
(8016 INFRACTION)

AS STATED IN YOUR DECEMBER 18, 1979 RESPONSE TO THE ITEM OF NONCOMPLIANCE DESCRIBED IN PARAGRAPH 7 OF REPORT 50-250/251/79-31, A MAINTENANCE INSTRUCTION DETAILING THE OPERATION OF THE SELECT-A-TORQ MACHINE WOULD BE ISSUED AND FULL COMPLIANCE WOULD BE ACHIEVED PRIOR TO MARCH 15, 1980. CONTRARY TO THE ABOVE, THIS INSTRUCTION WAS NOT ISSUED UNTIL APRIL 25, 1980, SUBSEQUENT TO THE RESIDENT INSPECTOR'S FOLLOWUP IN THIS AREA.
(8016 DEVIATION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

NORMAL OPERATION AT 100% POWER.

♦ LAST IE SITE INSPECTION DATE: JUNE 16-20, 1980

♦ INSPECTION REPORT NO: 50-250/80-23

SUMMARY

INSPECTION SUMMARY

♦ INSPECTION MAY 28-30 (80-19): THIS SPECIAL, ANNOUNCED INSPECTION INVOLVED 15 INSPECTOR-HOURS ON SITE IN THE AREAS OF INSTALLED EQUIPMENT REVIEW WITH RESPECT TO IE BULLETIN 79-01B. OF THE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION MAY 1-31 (80-21): THIS ROUTINE INSPECTION INVOLVED 60 INSPECTOR-HOURS ON SITE IN THE AREAS OF PLANT OPERATIONS, MAINTENANCE PROCEDURES, LICENSEE EVENT REPORT FOLLOWUP, PLANT TOURS AND SURVEILLANCE OF SAFETY-RELATED SYSTEMS AND COMPONENTS. OF THE FIVE AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN 3 AREAS; ONE APPARENT ITEM OF NONCOMPLIANCE WAS FOUND IN ONE AREA (INFRACTION - INADEQUATE MAINTENANCE PROCEDURE).

INSPECTION JUNE 11-13 (80-22): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 5.5 INSPECTOR-HOURS ON SITE IN THE AREAS OF FOLLOWUP ON IE BULLETINS, AND FOLLOWUP ON PREVIOUS INSPECTION FINDINGS. OF THE TWO AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

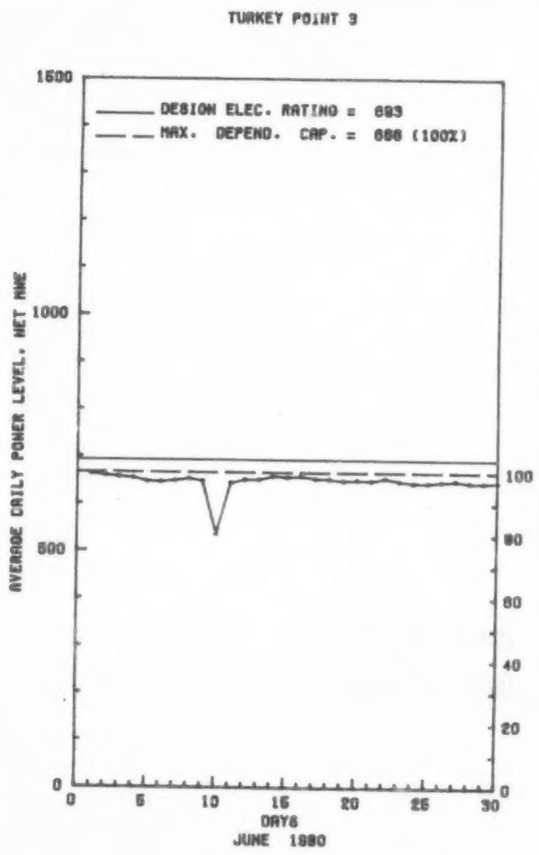
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-007/ 03L-0	04/24/80	05/26/80	SIX ELECTRICAL PENETRATION FIRE BARRIERS FOUND TO BE NON-FUNCTIONAL.
80-011/ 01T-0	05/27/80	06/10/80	ERROR IN ECCS ANALYSIS.

REVIEWED BY OMPA LOEB: ME Bole **DATE:** 8/4/80

REVIEWED BY NRR DL: Don Neighbors **DATE:** 8/5/80

TURKEY POINT 3

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-250
2. Utility Contact: V. T. Chilson (305) 552-3824
3. Licensed Thermal Power (MWt): 2200
4. Nameplate Rating (Gross MWe): 894 x 0.85 = 760
5. Design Electrical Rating (Net MWe): 693
6. Maximum Dependable Capacity (Gross MWe): 700
7. Maximum Dependable Capacity (Net MWe): 666
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

ITEM 21 CALCULATED BY USING A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	4,367.0	66,368.6
12. Hours Reactor Was Critical	719.4	3,491.8	50,761.2
13. Reactor Reserve Shutdown Hours	.0	.0	213.3
14. Hours Generator On-Line	717.0	3,334.7	49,024.9
15. Unit Reserve Shutdown Hours	.0	.0	121.8
16. Gross Thermal Energy (MWH)	1,566,925	7,185,007	98,653,641
17. Gross Electrical Energy (MWH)	489,820	2,273,405	31,354,405
18. Net Electrical Energy (MWH)	465,623	2,152,960	29,675,034
19. Unit Service Factor	99.6	76.4	73.9
20. Unit Availability Factor	99.6	76.4	74.1
21. Unit Capacity Factor (MDC Net)	97.1	74.0	67.8
22. Unit Capacity Factor (DER Net)	93.3	71.1	64.5
23. Unit Forced Outage Rate	.4	1.5	2.6

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): STEAM GEN. TUBE INSPECTION PROGRAM - OCT. 5 - NOV. 1, 1980.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
14	06/10/80	F	3.0	A	3		ED	GENERA	UNIT WAS TRIPPED BY STEAM GENERATOR NO. 3B LEVEL PROTECTION SYSTEM CAUSED BY THE LOSS OF POWER SUPPLY TO VITAL INSTRUMENT BUS NO. 3A. CORRECTIVE ACTIONS INCLUDED REPLACING SCRS IN INVERTER NO. A.

SUMMARY

THE UNIT RAN AT NEARLY 100% POWER FOR THE MONTH EXCEPT FOR A SCRAM ON 06/10/80 DUE TO LOSS OF POWER TO STEAM GENERATOR.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

best copy available

TURKEY POINT 4

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Florida/Dade 25 miles S of Miami
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2200
5.	Design Electrical Rating (MWE Net)	693
6.	Date of Initial Criticality	June 11, 1973
7.	Date of Commercial Operation	September 7, 1973
8.	Condenser Cooling Method	Recirculating Canal System
9.	Condenser Cooling Water Source	Atlantic Ocean

Utility & Contractor Information

10.	Licensee	Florida Power and Light Company
11.	Corporate Address	9250 West Flagler Street P. O. Box 013100 Miami, Florida 33101
12.	Corporate Contact	R. E. Uhrig, V. P. Nuclear Affairs
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. Lewis
18.	IE Section Chief	R. D. Martin
19.	IE Resident Inspector	R. J. Vogt-Lowell (305) 245-7669
20.	Licensing Project Manager	M. Grotenhuis (301) 492-7128
21.	Docket Number	50-251
22.	License Number/Date of Issue	DPR-41, 4/10/73
23.	Public Document Room	Environmental and Urban Affairs Library Florida International University Miami, Florida 33199

INSPECTION STATUS

ENFORCEMENT STATUS

AS REQUIRED BY 10 CFR 50, APPENDIX B, CRITERION XVI, AND TOPICAL QUALITY REQUIREMENT 16.0 OF THE TOPICAL QUALITY ASSURANCE REPORT NO. FPLTQAR 1-76, AS IMPLEMENTED IN PART BY PARAGRAPH 5.2.2.3 OF QUALITY PROCEDURE 16.1, CORRECTIVE ACTION FOR ITEMS OF NONCOMPLIANCE IDENTIFIED IN NRC INSPECTION REPORTS SHALL BE TRACKED BY THE PLANT. CONTRARY TO THE ABOVE, THE CORRECTIVE ACTION DESCRIBED IN YOUR DECEMBER 18, 1979 RESPONSE TO THE ITEM OF NONCOMPLIANCE DESCRIBED IN PARAGRAPH 7 OF REPORT 50-250/251/79-31 WAS NOT TRACKED BY THE PLANT THEREBY CONTRIBUTING TO THE LACK OF IMPLEMENTATION OF THE CORRECTIVE ACTION BY YOUR STATED DUE DATE.
(8014 INFRACTION)

AS STATED IN YOUR DECEMBER 18, 1979 RESPONSE TO THE ITEM OF NONCOMPLIANCE DESCRIBED IN PARAGRAPH 7 OF REPORT 50-250/251/79-31, A MAINTENANCE INSTRUCTION DETAILING THE OPERATION OF THE SELECT-A-TORQ MACHINE WOULD BE ISSUED AND FULL COMPLIANCE WOULD BE ACHIEVED PRIOR TO MARCH 15, 1980. CONTRARY TO THE ABOVE, THIS INSTRUCTION WAS NOT ISSUED UNTIL APRIL 25, 1980, SUBSEQUENT TO THE RESIDENT INSPECTOR'S FOLLOWUP IN THIS AREA.
(8014 DEVIATION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

PLANT OPERATING AT NEARLY FULL POWER.

+ LAST IE SITE INSPECTION DATE: JUNE 16-20, 1980

+ INSPECTION REPORT NO: 50-251/80-23

SUMMARY

INSPECTION SUMMARY

+ INSPECTION MAY 28-30 (80-19): THIS SPECIAL, ANNOUNCED INSPECTION INVOLVED 15 INSPECTOR-HOURS ON SITE IN THE AREAS OF INSTALLED EQUIPMENT REVIEW WITH RESPECT TO IE BULLETIN 79-01B. OF THE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION JUNE 4-6 (80-20): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 8.5 INSPECTOR-HOURS ON SITE IN THE AREAS OF FIRE PROTECTION/PREVENTION. OF THE AREAS INSPECTED, ONE ITEM OF NONCOMPLIANCE WAS FOUND (INFRACTION - INADEQUATE FIRE BRIGADE TRAINING PROGRAM; AND ONE DEVIATION WAS FOUND, DEVIATION - FIRE PROTECTION ADMINISTRATIVE CONTROL PROCEDURES DO NOT MEET THE NRC GUIDELINES).

INSPECTION MAY 1-31 (80-21): THIS ROUTINE INSPECTION INVOLVED 60 INSPECTOR-HOURS ON SITE IN THE AREAS OF PLANT OPERATIONS, MAINTENANCE PROCEDURES, LICENSEE EVENT REPORT FOLLOWUP, PLANT TOURS AND SURVEILLANCE OF SAFETY-RELATED SYSTEMS AND COMPONENTS. OF THE FIVE AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN 3 AREAS; ONE APPARENT ITEM OF NONCOMPLIANCE WAS FOUND IN ONE AREA (INFRACTION - INADEQUATE MAINTENANCE PROCEDURE).

INSPECTION JUNE 11-13 (80-22): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 5.5 INSPECTOR-HOURS ON SITE IN THE AREAS OF FOLLOWUP ON IE BULLETINS, AND FOLLOWUP ON PREVIOUS INSPECTION FINDINGS. OF THE TWO AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

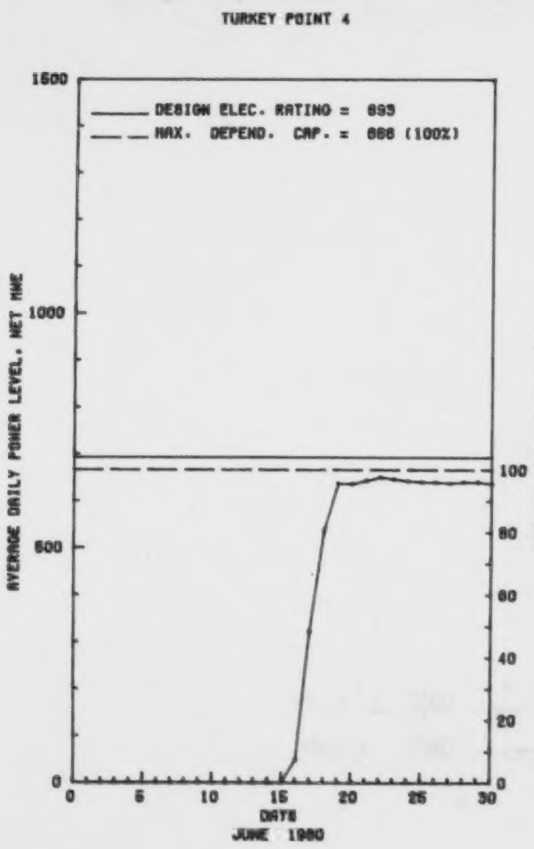
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-007/01T-0	05/07/80	05/21/80	DISC STUD MISSING FROM 4A MAINSTEAM CHECK VALVE.
80-008/01T-0	05/23/80	06/09/80	STEAM GENERATOR NOZZLE-TO-FEEDWATER LINE REDUCER HAS CRACKS.

REVIEWED BY OMPA LOEB: PE Bobe DATE: 8/4/80

REVIEWED BY NRR DL: Don Neighbors DATE: 8/5/80

TURKEY POINT 4

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-251
2. Utility Contact: V. T. Chilson (305) 552-3824
3. Licensed Thermal Power (MWt): 2200
4. Nameplate Rating (Gross MWe): 894 x 0.85 = 760
5. Design Electrical Rating (Net MWe): 693
6. Maximum Dependable Capacity (Gross MWe): 700
7. Maximum Dependable Capacity (Net MWe): 666
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

ITEM 21 CALCULATED BY USING A WEIGHTED AVERAGE.

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	4,367.0	60,396.0
12. Hours Reactor Was Critical	378.1	3,044.1	43,871.9
13. Reactor Reserve Shutdown Hours	.0	12.3	166.4
14. Hours Generator On-Line	340.7	2,956.8	42,139.3
15. Unit Reserve Shutdown Hours	.0	12.3	31.2
16. Gross Thermal Energy (MWH)	720,968	6,365,622	87,584,602
17. Gross Electrical Energy (MWH)	217,885	1,998,725	27,902,203
18. Net Electrical Energy (MWH)	203,927	1,891,641	26,421,637
19. Unit Service Factor	47.3	67.7	70.1
20. Unit Availability Factor	47.3	68.0	70.2
21. Unit Capacity Factor (MDC Net)	42.5	65.0	68.3
22. Unit Capacity Factor (DER Net)	40.9	62.5	63.4
23. Unit Forced Outage Rate	.8	.5	3.1

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
REFUEL, MAINT. & INSPECTIONS - NOV. 9, 1980 - JAN. 10, 1981.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
13	04/26/80	S	374.7	B	4		HB	TURBIN	UNIT WAS REMOVED FROM SERVICE TO PERFORM SCHEDULED MAINTENANCE AND INSPECTIONS, INCLUDING S/G TUBE INSPECTION PROGRAM, AND TURBINE L.P. ROTOR CHANGEOUT. (CONTINUED FROM PREVIOUS MONTH).
14	06/17/80	S	1.8	B	3		HB	INSTRU	UNIT WAS REMOVED FROM SERVICE TO TEST TURBINE OVERSPEED PROTECTION TRIP DEVICE. REACTOR WAS TRIPPED BY S/G NO. 4C LEVEL PROTECTION SYSTEM DURING A TRANSIENT CONDITION.
15	06/17/80	F	2.8	A	3		HA	TURBIN	UNIT WAS TRIPPED BY S/G NO. 4C LEVEL PROTECTION SYSTEM DURING A TRANSIENT CONDITION WHILE INCREASING LOAD AFTER THE UNIT WAS RETURNED TO SERVICE.

SUMMARY

THE UNIT RETURNED TO SERVICE ON 06/16/80 AFTER A MAINTENANCE OUTAGE AND OPERATED AT NEAR FULL POWER EXCEPT FOR TWO OUTAGES ON 06/17/80 FOR TURBINE OVERSPEED TRIP TEST AND BY S/G LEVEL PROTECTION SYSTEM.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

VERMONT YANKEE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Vermont/Windham 5 miles S of Brattleboro
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	1593
5.	Design Electrical Rating (MWE Net)	514
6.	Date of Initial Criticality	March 24, 1972
7.	Date of Commercial Operation	November 30, 1972
8.	Condenser Cooling Method	Closed Cycle Cooling Towers
9.	Condenser Cooling Water Source	Connecticut River

Utility & Contractor Information

10.	Licensee	Vermont Yankee Nuclear Power Corp.
11.	Corporate Address	20 Turnpike Road Westboro, Massachusetts 01581
12.	Corporate Contact	D. W. Edwards Assistant to the Vice President
13.	Architect/Engineer	Ebasco
14.	Nuclear Steam Supply System	GE
15.	Constructor	Ebasco

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	IE Section Chief	T. T. MARTIN
19.	IE Resident Inspector	W. J. RAYMOND (802) 257-4319
20.	Licensing Project Manager	V. L. Rooney
21.	Docket Number	50-271
22.	License Number/Date of Issue	DPR-28, 3/21/72
23.	Public Document Room	Brooks Memorial Library 224 Main Street Brattleboro, Vermont 05301

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO 10 CFR 50, APPENDIX B & APPROVED TEST PROCEDURE, REACTOR POWER, REACTOR PRESSURE & ACCUMULATOR PRESSURE WERE NOT ENTERED ON DATA FORM VYOP4424.01 FOR SCRAM NO. 94 OF IV19/79. (8006 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

- A routine drywell tour identified a bonnet leak on the "B" main feedwater inboard check valve. The plant was taken to cold shutdown for repairs.
- During routine return to power operations, Recirc M.G. "B" tripped as the operator was resetting relay targets for M.G. overcurrent sensing relays.
- The reactor automatically tripped from 89% power due to an indicated high level in the moisture separator drain tank. The cause was attributed to troubleshooting on the moisture separator emergency drain valve controls.

FACILITY ITEMS (PLANS AND PROCEDURES):

- Unit refueling outage scheduled to commence 9/26 is estimated to last approximately eight weeks.
- Vermont Yankee discharge permit renewal application is under review by the Vermont State Agency of Environmental Conservation. This permit authorizes VNPC to discharge controlled waste water from Vermont Yankee NPS into the Connecticut River.

MANAGERIAL ITEMS:

- A management meeting between the Region I staff and the licensee was held at the site on 6/27 to duscuss the result of the NRC's Systematic Licensee Performance Evaluation.
- A meeting between local public officials and NRC staff was conducted on 6/27 for the purpose of officially opening the Vermont Yankee Resident Inspector Office.

PLANT STATUS:

- Routine operations at rated power with plant cooling towers in service. Representative steam jet air ejector and stack release rates are 3300 uc/sec and less than 100 uc/sec, respectively.

LAST IE SITE INSPECTION DATE: 5/23/80

INSPECTION REPORT NO: 50-271/80-08

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-017/03L	04/24/80	05/23/80	PCIS valve FSO-109-76B failed to close during TS required quarterly test due to dirt buildup on valve plunger assembly
80-019/01T	05/16/80	05/30/80	Both advanced off gas system radiation monitors taken off line for 10 minutes to help evaluate temperature anomaly in AOG system

SUMMARY

INSPECTION SUMMARY

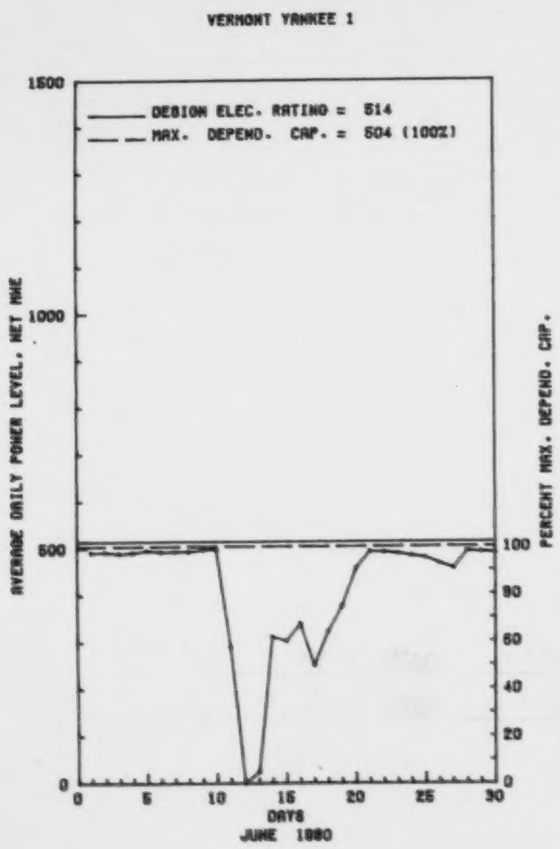
- 50-271/80-06 - Apr 28 - May 1: Routine unannounced inspection by one regional based inspector (31 hours) of plant staff training, requalification training for licensed operators, craft and technician training, and post refueling startup testing. One item of noncompliance was identified: Failure to record test data completely.
- 50-271/80-07 - May 19-23: Routine unannounced inspection by one regional based inspector (26 hours) of the plant Fire Protection/Prevention Program, including: implementation of administrative procedures; fire brigade training; observation of ignition source and combustible materials control; review and observation of plant modifications; inspection of critical plant areas identified in T.I. 2515/19; and, review of actions taken on previous inspection findings. No items of noncompliance were identified.

REVIEWED BY OMPA LOEB: DATE: 8-4-80

REVIEWED BY NRR DL: DATE: 8/4/80

VERMONT YANKEE

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-271
2. Utility Contact: F.C. BEERS (617) 366-9011
3. Licensed Thermal Power (MWt): 1593
4. Nameplate Rating (Gross MWe): 626 x 0.9 = 563
5. Design Electrical Rating (Net MWe): 514
6. Maximum Dependable Capacity (Gross MWe): 535
7. Maximum Dependable Capacity (Net MWe): 504
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	4,367.0	68,161.8
12. Hours Reactor Was Critical	672.2	4,173.5	55,332.1
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	660.0	4,118.6	53,446.4
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	969,742	6,309,188	75,533,799
17. Gross Electrical Energy (MWH)	318,668	2,141,802	25,111,672
18. Net Electrical Energy (MWH)	298,640	2,043,101	23,811,466
19. Unit Service Factor	91.7	94.3	78.4
20. Unit Availability Factor	91.7	94.3	78.4
21. Unit Capacity Factor (MDC Net)	82.3	92.8	69.3
22. Unit Capacity Factor (DER Net)	80.7	91.1	68.0
23. Unit Forced Outage Rate	8.3	2.1	6.2

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): CRD TESTING AS REQUIRED BY NRC, MID JULY. REFUEL, 09/26/80.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-14	06/11/80	F	51.5	A	1		CH	VALVEX	LEAKAGE FROM FEEDWATER CHECK VALVE CAUSED BY FAILED VALVE SEAL. REPLACED SEAL.
80-15	06/15/80	F	.0	A	5		CB	RELAYX	RECIRCULATION PUMP TRIPPED WHILE RESETTING PUMP MG SET RELAYS. RESTARTED PUMP.
80-16	06/17/80	F	8.5	A	3		HA	VALVEX	HIGH LEVEL IN MAIN TURBINE MOISTURE SEPARATOR DRAIN TANK TRIPPED THE TURBINE AND SCRAMMED THE REACTOR. HIGH LEVEL CAUSED BY FAULTY DRAIN VALVE. THE VALVE WAS REPAIRED.

SUMMARY

THE PLANT RAN AT FULL POWER EXCEPT FOR A REDUCTION DUE TO RECIRC PUMP TRIP, AND OUTAGES ON 06/11/80 FOR FEEDWATER CHECK VALVE REPAIR AND ON 05/17/80 DUE TO HIGH WATER LEVELS IN THE MOISTURE SEPARATOR DRAIN TANK.

LEGEND

[1] Type:
F–Forced
S–Scheduled

[2] Reason:
A–Equipment Failure
B–Maintenance or Test
C–Refueling
D–Regulatory Restriction
E–Operator Training & License Examination
F–Administrative
G–Operational Error
H–Other

[3] Method:
1–Manual
2–Manual Scram
3–Automatic Scram
4–Continuations
5–Load Reduction
9–Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

YANKEE ROWE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Massachusetts/Franklin 25 miles NE of Pittsfield
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	600
5.	Design Electrical Rating (MWE Net)	175
6.	Date of Initial Criticality	August 19, 1960
7.	Date of Commercial Operation	July 1961
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Deerfield River

Utility & Contractor Information

10.	Licensee	Yankee Atomic Electric Company
11.	Corporate Address	20 Turnpike Road Westboro, Massachusetts 01581
12.	Corporate Contact	D. W. Edwards, Asst. to V.P.
13.	Architect/Engineer	Stone & Webster
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Section Chief	T. T. MARTIN
19.	IE Resident Inspector	T. FOLEY (413)424-5242
20.	Licensing Project Manager	A. Burger
21.	Docket Number	50-29
22.	License Number/Date of Issue	DPR-3, 12/24/63
23.	Public Document Room	Greenfield Community College 1 College Drive Greenfield, Massachusetts 01301

INSPECTION STATUS

ENFORCEMENT STATUS

T.S. 3.5.1 REQUIRES A LOW PRESSURE SAFETY INJECTION ACCUMULATOR TO BE OPERABLE IN MODES 1, 2, 3, 4, 5 WHEN MAIN COOLANT PRESSURE IS GREATER THAN 1000#. DURING THE PERIOD NOVEMBER 4, 1979 TO JANUARY 19, 1980 YANKEE ROWE OPERATED WITH AN INOPERABLE ACCUMULATOR.
(8002 VIOLATION)

T/S 6.8.1 REQUIRES WRITTEN PROCEDURES TO BE ESTABLISHED, IMPLEMENTED AND MAINTAINED. YANKEE ROWE FAILED TO FOLLOW PROCEDURES IN THAT (1) SWITCHING AND TAGGING ORDER FORMS WERE NOT CHECKED OR REPORTED CLEAR AS REQUIRED BY IN-PLANT TAGGING PROCEDURES. (2) JUMPER CONTROL REQUEST FORMS WERE NOT MAINTAINED AS REQUIRED BY AP-0018 "BYPASS OF SAFETY FUNCTION AND JUMPER CONTROL."
(*902 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

- SYSTEMS AND COMPONENT PROBLEMS:
- The schedule for the completion of the Low Pressure Turbine inner cylinder repair has been extended until November 1980.

PLANT STATUS:

- Plant shutdown and cooled down with expected restart scheduled at the end of the year.
- LAST IE SITE INSPECTION DATE: 6/13/80
- INSPECTION REPORT NO: 50-29/80-11

SUMMARY

INSPECTION SUMMARY

- None.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

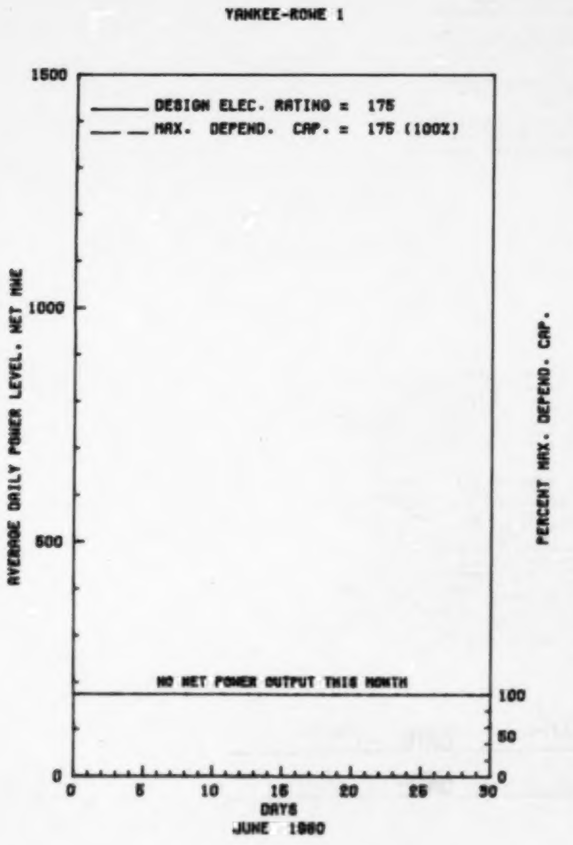
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
NONE				

REVIEWED BY OMPA LOEB: **DATE:** 8/4/80

REVIEWED BY NRR DL: **DATE:** 8/5/80

YANKEE ROWE

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-029
2. Utility Contact: F.C. BEERS (617) 366-9011
3. Licensed Thermal Power (MWt): 600
4. Nameplate Rating (Gross MWe): 185 x 1.0 = 185
5. Design Electrical Rating (Net MWe): 175
6. Maximum Dependable Capacity (Gross MWe): 180
7. Maximum Dependable Capacity (Net MWe): 175

CAPACITY FACTORS CALCULATED WITH A WEIGHTED AVERAGE

8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	4,367.0	172,004.0
12. Hours Reactor Was Critical	.0	445.8	138,810.4
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	.0	439.7	134,467.1
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	253,769	72,002,091
17. Gross Electrical Energy (MWH)	0	80,635	22,152,788
18. Net Electrical Energy (MWH)	0	75,749	20,741,893
19. Unit Service Factor	.0	10.1	78.2
20. Unit Availability Factor	.0	10.1	78.2
21. Unit Capacity Factor (MDC Net)	.0	9.9	68.9
22. Unit Capacity Factor (DER Net)	.0	9.9	68.9
23. Unit Forced Outage Rate	.0	86.8	3.6

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: NOVEMBER 1980

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
2	06/01/80	S	720.0	A	[illegible]				THE PLANT IS SHUTDOWN FOR TURBINE ROTOR REPAIR AND OTHER PLANT MAINTENANCE WORK.

SUMMARY

THE PLANT REMAINS SHUTDOWN FOR TURBINE ROTOR REPAIR AND OTHER MAINTENANCE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

ZION 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Illinois/Lake 40 miles N of Chicago
2.	Electric Reliability Council	Mid-America Interpool Network
3.	Type of Reactor	PWR
4.	Power Level (MWT)	3250
5.	Design Electrical Rating (MWE Net)	1040
6.	Date of Initial Criticality	June 19, 1973
7.	Date of Commercial Operation	December 31, 1973
8.	Condenser Cooling Method	Straight Through
9.	Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10.	Licensee	Commonwealth Edison Co.
11.	Corporate Address	P. O. Box 767 Chicago, Illinois
12.	Corporate Contact	D. L. Peoples, Manager of Nuclear Licensing
13.	Architect/Engineer	Sargent & Lundy
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Various

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	R. L. Spessard
19.	IE Resident Inspector	J. E. Kohler (312) 746-2313
20.	Licensing Project Manager	E. Reeves
21.	Docket Number	50-295
22.	License Number/Date of Issue	DPR-39 issued 4/6/73
23.	Public Document Room	Zion-Benton Public Library 2600 Emmens Avenue Zion, Illinois 60099

INSPECTION STATUS

ENFORCEMENT STATUS

TECHNICAL SPECIFICATION 3.9.5D REQUIRES THAT POSITIVE REACTIVITY CHANGES SHALL NOT BE MADE BY BORON DILUTION UNLESS THE REACTOR IS SUBCRITICAL BY AT LEAST 10% DELTA K/K WHEN CONTAINMENT INTEGRITY IS NOT INTACT. CONTRARY TO THE ABOVE, ON DECEMBER 4, 1979, UNIT 1 REACTOR COOLANT SYSTEM (RCS) WAS DILUTED TO LESS THAN 10% SHUTDOWN WITHOUT CONTAINMENT INTEGRITY INTACT.
(8003 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

FACILITY ITEMS

- None.

MANAGERIAL ITEMS

- D. Howard has become the new Rad-Chem Supervisor responsible to G. Plime, Administrative Assistant to the Plant Superintendent. (80-11)

PLANT STATUS

Unit 1 operating routinely.

- LAST IE SITE INSPECTION DATE: May 27-29, 1980

- INSPECTION REPORT NO: 80-11

SUMMARY

INSPECTION SUMMARY

- Inspection on April 21; May 15-20, and 30-31, (80-08): Special inspection of Bergen-Paterson large bore hydraulic snubbers installed on Unit 2 steam generators. The inspection involved 60 inspectors onsite by one NRC inspector. No items of noncompliance or deviations were identified.

- Inspection on May 12-15 and 19-20, (80-09): Included a review of the Security Plan; Security Program Audit; Security Organization; Protected Area Physical Barriers; Vital Area Physical Barriers; Lighting; Access Control (Identification, Authorization, and Badging); Access Control (Search); Access Control (Escorting); Access Control (Vital Areas); Alarm Stations; Detection Aids; Assessment Aids; Communications; Testing and Maintenance; Compensatory Measure; Power Supply; Response; Locks, Keys and Combinations; Records and Reports; Licensee Action of Previous Inspection Findings; and Independent Inspection Effort. The inspection involved 103.5 inspector hours onsite by three NRC inspectors. The inspection began during the day shift; 22.0 inspection hours were accomplished during offshift periods. One unit of the plant is currently in a refueling outage. The licensee was found to be in compliance with NRC requirements within 20 areas examined during the inspection except for the following 5 items: Access Control (Identification, Authorization, and Badging) - The licensee did not conduct proper personnel screening in accordance with security plan commitments; Access Control (Vital Areas) - Several vital area doors were found unsecured and unattended; Access Control (Search) - The licensee failed to properly search several individuals and one vehicle prior to their entry into the protected area; Testing and Maintenance - The licensee failed to test several segments of an alarm system; and Communications - The licensee failed to adequately protect one communication system. As a result of Items 1 and 2 noted above, an Immediate Action Letter was issued to the licensee on May 23, 1980.

- Inspection on May 13-14, (80-10): Special, announced inspection of utility management and technical competence in the areas of: offsite staff; resources amoung utilities; organization arrangement for normal and accident situations; training and requalification of offsite staff; financial capability; emergency procedures; reporting of unusual events and management involvement; and overtime restrictions. The inspection involved 63 inspector hours at the corporate office by four RIII inspectors and three NRR personnel. No items of noncompliance or deviations were identified.

Inspection on May 27-29, (80-11): Routine, unannounced inspection of environmental protection of Unit 1 including: administrative and procedural controls; licensee internal audits; implementation and results of the radiological and nonradiological environmental monitoring programs; review of meteorological program; review of corrective actions from a previous inspection; review of licensee environmental event reports; and quality assurance/quality control of confirmatory measurements program for the nonradiological chemistry program. The inspection involved 13 inspector hours onsite by one NRC inspector. Of the six areas inspected, no apparent items of noncompliance or deviations were found in four areas; one apparent item of noncompliance was identified in one area (excess pH of waste water released to the environment) and three apparent items of noncompliance in a second area (failure to standardize sodium hydroxide solution during boric acid titration; failure to conduct the required action conductivity measurements; and failure to provide documentation for gas bottle surveillance).

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

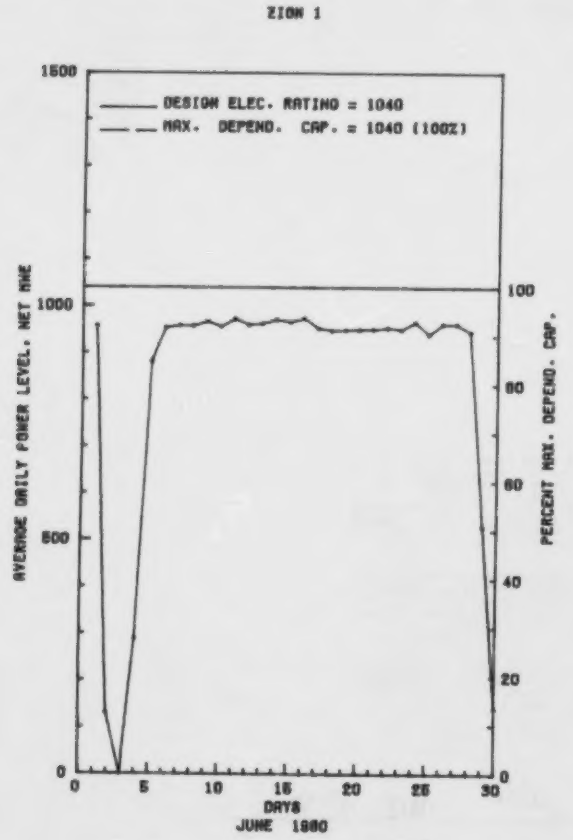
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-22/ 03L-0	05/07/80	06/06/80	The output signal from RC narrow range RTD, 1TE-421A intermittently spiking low.
80-23/ 01T-0	05/23/80	06/[illegible]/80	Discovery of existing charging pump 10 CFR Part 21 concern through Westinghouse generic notification and Zion site specific evaluation.
80-24/ 03L-0	05/15/80	06/13/80	During power operation while performing PT-10, 1A service water pump failed to start from train A of Safeguards.
80-25/ 03L-0	05/10/80	06/09/80	While performing periodic tests, maintenance personnel found function generator 1NM-441B output out of tolerance low 2.3%.
80-27/ 01T-0	06/01/80	06/12/80	While in cold shutdown testing of Unit 2 steam generator snubbers indicated that bleedrates under rated load were consistently lower than minimum of 0.116 in/min.

REVIEWED BY OMPA LOEB: ________ **DATE:** 8/4/80

REVIEWED BY NRR DL: ________ **DATE:** 8/5/80

ZION 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-295
2. Utility Contact: J. COOK (312) 746-2084
3. Licensed Thermal Power (MWt): 3250
4. Nameplate Rating (Gross MWe): 1220 x 0.9 = 1098
5. Design Electrical Rating (Net MWe): 1040
6. Maximum Dependable Capacity (Gross MWe): 1085
7. Maximum Dependable Capacity (Net MWe): 1040
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	4,367.0	56,975.0
12. Hours Reactor Was Critical	654.6	3,178.3	40,286.0
13. Reactor Reserve Shutdown Hours	.0	.0	2,621.8
14. Hours Generator On-Line	644.7	3,076.6	39,163.9
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	2,029,690	9,451,260	108,536,261
17. Gross Electrical Energy (MWH)	627,220	2,944,935	35,176,985
18. Net Electrical Energy (MWH)	598,250	2,791,346	33,275,592
19. Unit Service Factor	89.5	70.5	68.7
20. Unit Availability Factor	89.5	70.5	68.7
21. Unit Capacity Factor (MDC Net)	79.9	61.5	56.2
22. Unit Capacity Factor (DER Net)	79.9	61.5	56.2
23. Unit Forced Outage Rate	10.5	29.5	14.7

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
10	06/01/80	F	.0	A	5				LOAD REDUCED FOR HEAT TRACE INOPERABILITY.
11	06/02/80	F	.1	A	1				TURBINE OFF-LINE DUE TO S/G SNUBBER INOPERABILITY.
12	06/02/80	F	3.0	A	3				REACTOR TRIP DUE TO INSTRUMENT MALFUNCTION.
13	06/02/80	F	49.8	A	4				TURBINE OFF-LINE DUE TO S/G SNUBBER INOPERABILITY FOLLOWING REACTOR TRIP.
14	06/29/80	F	.0	A	5				LOAD REDUCED FOR THE PURPOSE OF LOCATING LEAK IN THE CONTAINMENT.
15	06/29/80	F	22.4	A	1				UNIT TAKEN OFF-LINE TO REPAIR COMPONENT COOLING LEAK ON REACTOR COOLANT PUMP 1B.

SUMMARY

UNIT 1 OPERATED NEAR FULL POWER DURING MOST OF JUNE, EXCEPT FOR REDUCTIONS FOR HEAT TRACE AND SNUBBER MALFUNCTION, AND LEAK REPAIR. TWO OUTAGES OCCURRED ON 06/02/80 AND 06/29/80 DUE TO INSTRUMENT MALFUNCTION AND REACTOR PUMP COMPONENT COOLANT LEAK.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

ZION 2

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Illinois/Lake 40 miles N of Chicago
2.	Electric Reliability Council	Mid-America Interpool Network
3.	Type of Reactor	PWR
4.	Licensee Power Level (MWT)	3250
5.	Design Electrical Rating (MWE Net)	1040
6.	Date of Initial Criticality	December 24, 1973
7.	Date of Commercial Operation	September 17, 1974
8.	Condenser Cooling Method	Straight Through
9.	Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10.	Licensee	Commonwealth Edison Co.
11.	Corporate Address	P. O. Box 767 Chicago, Illinois
12.	Corporate Contact	D. L. Peoples, Manager of Nuclear Licensing
13.	Architect/Engineer	Sargent & Lundy
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Various

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	R. L. Spessard
19.	IE Resident Inspector	J. E. Kohler (312) 746-2313
20.	Licensing Project Manager	E. Reeves
21.	Docket Number	50-304
22.	License Number/Date of Issue	DPR-48, 11/14/73
23.	Public Document Room	Zion-Benton Public Library 2600 Emmans Avenue Zion, Illinois 60099

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENTS

- The steam generator seismic snubbers have been tested and are being modified during the outage.

FACILITY ITEMS

- A successful integrated leak rate test was completed during this refueling outage.

MANAGERIAL ITEMS

- Mr. N. Wandke, Station Superintendent, has been assigned to Purchasing. Mr. K. Graesser, has been assigned to replace Mr. Wandke as Zion Station Superintendent. Mr. K. Kofron, has replaced Mr. E. Murach as Assistant Superintendent of Maintenance. (80-07)
- D. Howard has become the new Rad-Chem Supervisor responsible to G. Plime, Administrative Assistant to the Plant Superintendent. (80-11)

PLANT STATUS

- Unit began a routine refueling outage on May 2, 1980. The outage has been extended from 56 to 69 days due to additional seismic rework as a result of IE Bulletin 79-14, and repairs to the low pressure turbine.

+LAST IE SITE INSPECTION DATE: May 27-29, 1980

+ INSPECTION REPORT NO: 80-11

SUMMARY

INSPECTION SUMMARY

- Inspection on April 29-30; May 19-21; and June 3-4, (80-07): Review of the ISI program and procedures for this refueling outage, observation of ISI examination activities, and review of examination results. The inspection involved 51 inspector hours onsite by one NRC inspector. No items of noncompliance or deviations were identified.
- Inspection on April 21; May 15-20, and 30-31, (80-08): Special inspection of Bergen-Paterson large bore hydraulic snubbers installed on Unit 2 steam generators. The inspection involved 60 inspectors onsite by one NRC inspector. No items of noncompliance or deviations were identified.
- Inspection on May 12-15 and 19-20, (80-09): Included a review of the Security Plan; Security Program Audit; Security Organization; Protected Area Physical Barriers; Vital Area Physical Barriers; Lighting; Access Control (Identification, Authorization, and Badging); Access Control (Search); Access Control (Escorting); Access Control (Vital Areas); Alarm Stations; Detection Aids; Assessment Aids; Communications; Testing and Maintenance; Compensatory Measure; Power Supply; Response; Locks, Keys and Combinations; Records and Reports; Licensee Action of Previous Inspection Findings; and Independent Inspection Effort. The inspection involved 103.5 inspector hours onsite by three NRC inspectors. The inspection began during the day shift; 22.0 inspection hours were accomplished during offshift periods. One unit of the plant is currently in a refueling outage. The licensee was found to be in compliance with NRC requirements within 20 areas examined during the inspection except for the following 5 items: Access Control (Identification, Authorization, and Badging) - The licensee did not conduct proper personnel screening in accordance with security plan commitments; Access Control (Vital Areas) - Several vital area doors were found unsecured and unattended; Access Control (Search) - The licensee failed to properly search several individuals and one vehicle prior to their entry into the protected area; Testing and Maintenance - The licensee failed to test several segments of an alarm system; and Communications - The licensee failed to adequately protect one communication system. As a result of Items 1 and 2 noted above, an Immediate Action Letter was issued to the licensee on May 23, 1980.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-19/ 03L-0	04/29/80	05/27/80	During Safeguards Actuation Test, 2A containment spray pump stop valve 2MOV-CS0002 failed to open.
80-20/ 03L-0	05/07/80	06/03/80	During refuel. maintenance, S/G 2A pres. channel was found low by 2.6% exceeding channel accuracy (plus or minus 2%).

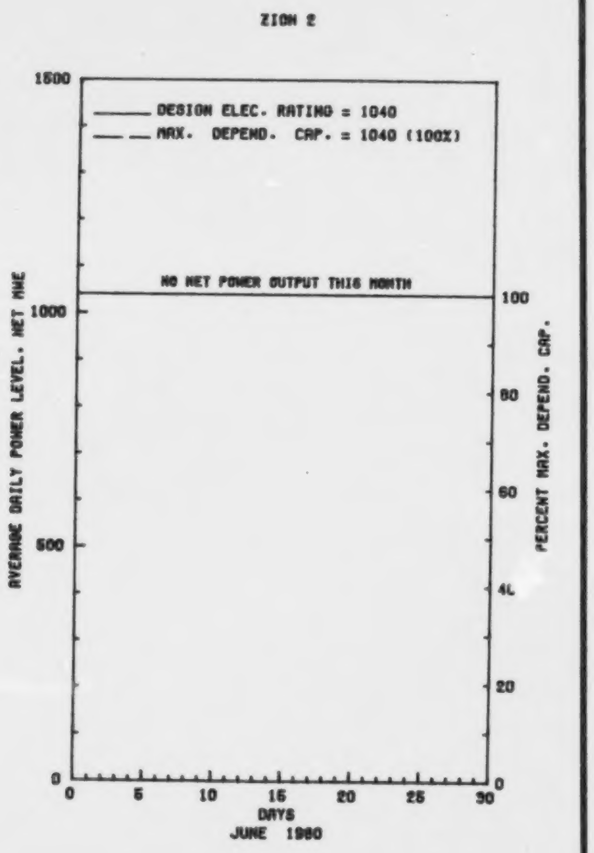
- Inspection on May 13-14, (80-10): Special, announced inspection of utility management and technical competence in the areas of: offsite staff; resources amoung utilities; organization arrangement for normal and accident situations; training and requalification of offsite staff; financial capability; emergency procedures; reporting of unusual events and management involvement; and overtime restrictions. The inspection involved 63 inspector hours at the corporate office by four RIII inspectors and three NRR personnel. No items of noncompliance or deviations were identified.
- Inspection on May 27-29, (80-11): Routine, unannounced inspection of environmental protection of Unit 2 including: administrative and procedural controls; licensee internal audits; implementation and results of the radiological and nonradiological environmental monitoring programs; review of meteorological program; review of corrective actions from a previous inspection; review of licensee environmental event reports; and quality assurance/quality control of confirmatory measurements program for the nonradiological chemistry program. The inspection involved 18 inspector hours onsite by one NRC inspector. Of the six areas inspected, no apparent items of noncompliance or deviations were found in four areas; one apparent item of noncompliance was identified in one area (excess pH of waste water released to the environment) and three apparent items of noncompliance in a second area (failure to standardize sodium hydroxide solution during boric acid titration; failure to conduct the required action conductivity measurements; and failure to provide documentation for gas bottle surveillance).

REVIEWED BY OMPA LOEB: P E Bobe **DATE:** 8/4/80

REVIEWED BY NRR DL: E Reeves **DATE:** 8/5/80

ZION 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-304
2. Utility Contact: J. COOK (312) 746-2084
3. Licensed Thermal Power (MWt): 3250
4. Nameplate Rating (Gross MWe): 1220 x 0.9 = 1098
5. Design Electrical Rating (Net MWe): 1040
6. Maximum Dependable Capacity (Gross MWe): 1085
7. Maximum Dependable Capacity (Net MWe): 1040
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	4,367.0	50,688.0
12. Hours Reactor Was Critical	.0	2,439.0	36,042.6
13. Reactor Reserve Shutdown Hours	.0	.0	226.1
14. Hours Generator On-Line	.0	2,409.1	35,223.0
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	7,382,048	100,017,366
17. Gross Electrical Energy (MWH)	0	2,417,965	32,139,185
18. Net Electrical Energy (MWH)	0	2,296,176	30,497,634
19. Unit Service Factor	.0	55.2	69.5
20. Unit Availability Factor	.0	55.2	69.5
21. Unit Capacity Factor (MDC Net)	.0	50.6	57.9
22. Unit Capacity Factor (DER Net)	.0	50.6	57.9
23. Unit Forced Outage Rate	.0	17.8	17.7

24. Shutdowns Scheduled Over Next 6 Months (Type,Date,and Duration of Each): NONE
25. If Shut Down At End of Report Period Estimated Date of Startup: 07/11/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
12	06/02/80	S	720.0	C	4		RC	FUELXX	CONTINUATION OF MAY REFUELING OUTAGE.

SUMMARY

THE PLANT WAS SHUTDOWN THE ENTIRE MONTH FOR REFUELING.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

SECTION 3 – APPENDIX

STATUS OF SPENT FUEL STORAGE CAPABILITY

DATA AS OF 6-30-80

Facility	Core Size [a] (No. of Assemblies)	Present Authorized Storage Pool Capacity (Fuel Assemblies)	No. of Assemblies Stored	Remaining Capacity (No. of Assemblies)	Remaining Capacity If Pending Request Approved (No. of Assemblies)	Scheduled Date For Next Refueling	Will Fill Present [b] Authorized Capacity
I. Pressurized Water Reactors							
ARKANSAS 1	177	590	176	414		01-81	1988
ARKANSAS 2	177	486	58	428		03-81	1989
BEAVER VALLEY 1	157	833	0	833		N/S	1995
CALVERT CLIFFS 1	217	1056(c)	364(c)	692(c)		10-80	1984
CALVERT CLIFFS 2	217					01-81	
COOK 1	193	500	193	307	1857	N/S	1985
COOK 2	193					N/S	
CRYSTAL RIVER 3	177	256	112	144	1109	N/S	1982
DAVIS-BESSE 1	177	735	52	683		N/S	1989
FARLEY 1	157	675	0	675		09-80	1990
FORT CALHOUN 1	133	483	197	286		03-81	1985
GINNA	121	595	156	439		N/S	1987
HADDAM NECK	157	1166	389	779		09-81	1994
INDIAN POINT 1	0	288	160	128		N/S	
INDIAN POINT 2	193	482	200	282		N/S	1984
INDIAN POINT 3	193	837	64	773		N/S	1989
KEWAUNEE	121	990	120	870		N/S	1999
MAINE YANKEE	217	953	433	520		N/S	1986
MILLSTONE 2	217	667	144	523		08-80	1987
NORTH ANNA 1	157	400	0	400	966	N/S	1984
NORTH ANNA 2(d)	157	400	0	400		N/S	
OCONEE 1	177	1224(l)	737	487(l)	755	05-81	1980
OCONEE 2	177					06-81	
OCONEE 3	177					12-80	
PALISADES	204	798	273	525		N/S	1985
POINT BEACH 1	121	1502(c)	276(c)	1226(c)		11-80	1995
POINT BEACH 2	121					03-81	
PRAIRIE ISLAND 1	121	687(c)	280(c)	407(c)		08-80	1984
PRAIRIE ISLAND 2	121					02-81	
RANCHO SECO 1	177	579	164	415		07-81	1987
ROBINSON 2	157	272	106	166(e)		07-80	1980(g)
SALEM 1	193	264	40	224	1130	09-80	1982
SAN ONOFRE 1	157	216	97	119		N/S	1983
ST LUCIE 1	217	728	60	668		N/S	1990
SURRY 1	157	1044(c)	1983(c)	608(c)		09-80	1983
SURRY 2	157					09-80	
THREE MILE ISLAND 1	177	752	208	544		N/S	1989
THREE MILE ISLAND 2	177	442	0	442		N/S	1986
TROJAN	193	651	128	587		04-81	1988
TURKEY POINT 3	157	621	346(c)	275(c)		N/S	1987
TURKEY POINT 4	157					N/S	
YANKEE-ROWE 1	76	391	149	242	572	N/S	1985
ZION 1	193	2112(c)	508(c)	1604(c)	1676	01-81	1993
ZION 2	193					N/S	

Facility	Core Size [a] (No. of Assemblies)	Present Authorized Storage Pool Capacity (Fuel Assemblies)	No. of Assemblies Stored	Remaining Capacity (No. of Assemblies)	Remaining Capacity If Pending Request Approved (No. of Assemblies)	Scheduled Date For Next Refueling	Will Fill Present [b] Authorized Capacity
II. Boiling Water Reactors							
BIG ROCK POINT 1	84	193	88	105	353	10-80	1983
BROWNS FERRY 1	764	3471	576	2795		04-81	1985
BROWNS FERRY 2	764	1080	464	616		09-80	1985
BROWNS FERRY 3	764	1528	352	1176		10-80	1985
BRUNSWICK 1	560	154PWR+476BWR (f)	(k)			N/S	1986
BRUNSWICK 2	560	105PWR+264BWR(f)	91PWR+132BWR			N/S	1986
COOPER STATION	548	2366	284	2082		N/S	1995
DRESDEN 1	464	672	221	451		N/S	1990
DRESDEN 2	724	2840(c)	1069(c)	1771(c)	6491(c)	N/S	1985
DRESDEN 3	724					N/S	
DUANE ARNOLD	368	2050	384	1666		04-81	1998
FITZPATRICK	560	760	268	492	1752	N/S	1981
HATCH 1	560	840	260	580		12-80	1988
HATCH 2	560	1120	0	1120		10-80	
HUMBOLDT BAY	172	487	251	236		N/S	
LA CROSSE	72	134	113	21	327	09-80	1980
MILLSTONE 1	580	2184	776	1408		09-80	1991
MONTICELLO	484	2237	828	1409		N/S	1991
NINE MILE POINT 1	532	1984	660	1324	2349	N/S	1990
OYSTER CREEK 1	560	1800	780	1020		N/S	1987
PEACH BOTTOM 2	764	2816	910	1906		N/S	1990
PEACH BOTTOM 3	764	2816	712	2104		03-81	1991
PILGRIM 1	580	2320	764	1556		09-81	1990
QUAD CITIES 1	724	1460	596	864		09-80	1986
QUAD CITIES 2	724	1460	492	968		N/S	1985
VERMONT YANKEE 1	368	2000	894	1106		09-80	1988
III. Independent Spent Fuel Storage Installations (h)							
Morris Operations		750 MTU (j)	315	385 MTU (j)	1490 MTU (j)		
NFS(i)		250 MTU	170 MTU	80 MTU			

(a) At each refueling outage approximately 1/3 of a PWR core and 1/4 of a BWR core is off-loaded.
(b) Some of these dates have been adjusted by staff assumptions.
(c) This is the total for both units.
(d) Plant not in commercial operation.
(e) Some spent fuel stored at Brunswick.
(f) Authorized a total 2088 BWR and 304 PWR assemblies for both pools.
(g) Robinson 2 assemblies being shipped to Brunswick for storage.
(h) Capacity is in metric tons of uranium; 1 MTU = 2 PWR assemblies or 5 BWR assemblies.
(i) No longer accepting spent fuel.
(j) Racked for 700 MTU.
(k) Reserved
(l) This is the station total.

N/S = Not Scheduled

REACTOR-YEARS OF EXPERIENCE

AS OF 6-30-80

The total of Reactor years of experience is as the sum of all calendar days for each unit, from the date that electricity was first generated until a final shutdown date or the status date, whichever comes first, divided by 365.25 days/year.

If a date is unknown, the first day of the first month of operation is substituted. Units which have not yet generated electricity but which are licensed are listed but not included in the computation.

LICENSED OPERATING ELECTRICAL PRODUCING UNITS

UNIT NAME	FIRST ELEC. GENERATION	YEARS EXPERIENCE
ARKANSAS 1	08/31/74	5.92
ARKANSAS 2	12/26/78	1.51
BEAVER VALLEY 1	06/14/76	4.05
BIG ROCK POINT 1	12/08/62	17.56
BROWNS FERRY 1	10/15/73	6.71
BROWNS FERRY 2	08/28/74	5.84
BROWNS FERRY 3	09/12/76	3.80
BRUNSWICK 1	12/04/76	3.57
BRUNSWICK 2	04/29/75	5.17
CALVERT CLIFFS 1	01/03/75	5.49
CALVERT CLIFFS 2	12/07/76	3.56
COOK 1	02/10/75	5.38
COOK 2	03/22/78	2.28
COOPER STATION	05/10/74	6.14
CRYSTAL RIVER 3	01/30/77	3.42
DAVIS-BESSE 1	08/28/77	2.84
DRESDEN 1	04/15/60	20.21
DRESDEN 2	04/13/70	10.22
DRESDEN 3	07/22/71	8.94
DUANE ARNOLD	05/19/74	6.12
FARLEY 1	08/18/77	2.87
FITZPATRICK	02/01/75	5.41
FORT CALHOUN 1	08/25/73	6.85
FORT ST VRAIN	12/11/76	3.55
GINNA	12/02/69	10.58
HADDAM NECK	08/07/67	12.90
HATCH 1	11/11/74	5.64
HATCH 2	09/22/78	1.77
INDIAN POINT 2	06/26/73	7.01
INDIAN POINT 3	04/27/76	4.18
KEWAUNEE	04/08/74	6.23
LA CROSSE	04/26/68	12.18
MAINE YANKEE	11/08/72	7.64
MILLSTONE 1	11/29/70	9.59
MILLSTONE 2	11/09/75	4.64
MONTICELLO	03/05/71	9.33
NINE MILE POINT 1	11/09/69	10.64
NORTH ANNA 1	04/17/78	2.21
OCONEE 1	05/06/73	7.15
OCONEE 2	12/05/73	6.57
OCONEE 3	09/01/74	5.83
OYSTER CREEK 1	09/23/69	10.77
PALISADES	12/31/71	8.50
PEACH BOTTOM 2	02/18/74	6.37
PEACH BOTTOM 3	09/01/74	5.83
PILGRIM 1	07/19/72	7.95
POINT BEACH 1	11/06/70	9.65
POINT BEACH 2	08/02/72	7.91
PRAIRIE ISLAND 1	12/04/73	6.57
PRAIRIE ISLAND 2	12/21/74	5.53
QUAD CITIES 1	04/12/72	8.22
QUAD CITIES 2	05/23/72	8.11
RANCHO SECO 1	10/13/74	5.72
ROBINSON 2	09/26/70	9.76
SALEM 1	12/25/76	3.52

LICENSED OPERATING ELECTRICAL PRODUCING UNITS - CONT.

UNIT NAME	FIRST ELEC. GENERATION	YEARS EXPERIENCE
SAN ONOFRE 1	07/16/67	12.96
ST LUCIE 1	05/07/76	4.15
SURRY 1	07/04/72	7.99
SURRY 2	03/10/73	7.31
THREE MILE ISLAND 1	06/19/74	6.03
TROJAN	12/23/75	4.52
TURKEY POINT 3	11/02/72	7.66
TURKEY POINT 4	06/21/73	7.03
VERMONT YANKEE 1	09/20/72	7.78
YANKEE-ROWE 1	11/10/60	19.64
ZION 1	06/28/73	7.01
ZION 2	12/26/73	6.51
TOTAL		468.79 YEARS

PERMANENTLY OR INDEFINITELY SHUTDOWN UNITS

UNIT NAME	FIRST ELEC. GENERATION	SHUTDOWN OR ELECTRICAL CESSATION DATE	YEARS EXPERIENCE
BONUS	08/14/64	06/01/68	3.80
CVTR	12/18/63	01/01/67	3.04
ELK RIVER	08/24/63	02/01/68	4.44
FERMI 1	08/05/66	11/29/72	6.32
HALLAM	05/29/63	09/01/64	1.26
HUMBOLDT BAY	04/18/63	07/02/76	13.21
INDIAN POINT 1	09/16/62	10/31/74	12.12
PATHFINDER	07/25/66	10/01/67	1.19
PEACH BOTTOM 1	01/27/67	11/01/74	7.76
PIQUA	11/04/63	01/01/66	2.16
THREE MILE ISLAND 2	04/21/78	03/28/79	.93
TOTAL			56.23 YEARS

NON–POWER REACTORS IN THE U.S.

1. RESEARCH REACTORS

State	Location	Licensee	Reactor Type	Docket	License Number	Date Issued	Authorized Power Level (KW)
Alabama	1. Tuskegee	Tuskegee Institute	AGN 201 No. 102	50-406	R-122	08-30-74	.0001
Arizona	2. Tucson	U. of Arizona	TRIGA Mk. I	50-113	R-52	12-05-58	100
California	3. Los Angeles	U. of California	Argonaut	50-142	R-71	10-03-60	100
	4. Berkeley	U. of California	TRIGA Mark III	50-224	R-101	08-10-66	1000
	5. Irvine	U. of California	TRIGA Mark I	50-326	R-116	11-24-69	250
	6. Santa Barbara	U. of California	L-77	50-433	R-124	12-03-74	.01
	7. Canoga Park	Rockwell International Corporation	L-77	50-84	R-40	05-17-58	.01
	8. Hawthorne	Northrop Corporate Laboratories	TRIGA Mark F	50-187	R-90	03-04-63	1000
	9. San Luis Obispo	California State Polytechnic College	AGN 201 No. 100	50-394	R-121	05-16-73	.0001
	10. Alameda County	General Electric Co.	NTR	50-73	R-33	10-31-57	100
	11. San Diego	General Atomic Co.	TRIGA Mark I	50-89	R-38	05-03-58	250
	12. San Diego	General Atomic Co.	TRIGA Mark F	50-163	R-67	07-01-60	1500
	13. San Ramon	Aerotest Operations, Inc.	TRIGA (Industrial)	50-228	R-98	07-02-65	250
Colorado	14. Jefferson County	U.S. Geological Survey. Dept. of Interior	TRIGA Mark I	50-274	R-113	02-24-69	1000
Delaware	15. Newark	U. of Delaware	AGN-201 No. 113	50-98	R-43	07-03-58	.0001
District of Columbia	16. –	Catholic U.	AGN-201 No. 101	50-77	R-31	11-15-67	.0001
Florida	17. Gainesville	U. of Florida	Argonaut	50-83	R-56	05-21-59	100
Georgia	18. Atlanta	Georgia Institute of Technology	AGN-201 No. 104	50-276	R-111	04-19-68	.0001
	19. Atlanta	Georgia Institute of Technology	Heavy Water	50-160	R-97	12-29-64	5000
Idaho	20. Pocatello	Idaho State U.	AGN-201 No. 103	50-284	R-110	10-11-67	.0001
Illinois	21. Urbana	U. of Illinois	Advanced TRIGA	50-151	R-115	07-22-69	1500
	22. Urbana	U. of Illinois	LOPRA	50-356	R-117	12-27-71	10
	23. Zion	Westinghouse Electric Corporation	NTR	50-87	R-119	01-28-72	10
Indiana	24. West Lafayette	Purdue U.	Lockheed	50-182	R-87	08-16-62	10

RESEARCH REACTORS – CONT.

State	Location	Licensee	Reactor Type	Docket	License Number	Date Issued	Authorized Power Level (KW)
Iowa	25. Ames	Iowa State U.	UTR-10	50-116	R-59	10-16-59	10
Kansas	26. Lawrence	U. of Kansas	Lockheed	50-148	R-78	06-23-61	250
	27. Manhattan	Kansas State U.	TRIGA	50-188	R-88	10-16-62	250
Maryland	28. College Park	U. of Maryland	TRIGA	50-166	R-70	10-14-60	250
	29. Bethesda	Armed Forces Radiobiology Research Institute	TRIGA	50-170	R-84	06-26-62	1000
Massachusetts	30. Lowell	U. of Lowell	GE	50-223	R-125	12-24-74	1000
	31. Cambridge	Massachusetts Institute of Technology	Heavy Water Reflected	50-20	R-37	06-09-58	5000
	32. Worcester	Worcester Polytechnic Institute	GE	50-134	R-61	12-16-59	10
Michigan	33. Ann Arbor	U. of Michigan	Pool	50-2	R-28	09-13-57	2000
	34. Lansing	Michigan State U.	TRIGA Mark I	50-294	R-114	03-21-69	250
	35. Midland	Dow Chemical Co.	TRIGA Mark I	50-264	R-108	07-03-67	100
Missouri	36. Rolla	U. of Missouri	Pool	50-123	R-79	11-21-61	200
	37. Columbia	U. of Missouri	Tank	50-186	R-103	10-11-66	10,000
Nebraska	38. Omaha	V.A. Hospital	TRIGA	50-131	R-57	06-26-59	18
New Mexico	39. Albuquerque	U. of New Mexico	AGN-201M No. 112	50-252	R-102	09-17-66	.005
New York	40. Buffalo	State U. of New York	PULSTAR	50-57	R-77	03-24-61	2000
	41. Sterling Forest	Union Carbide Corp.	Pool	50-54	R-81	09-07-61	5000
	42. New York City	Columbia U.	TRIGA Mark II	50-208	R-128	04-14-77	250
	43. Ithaca	Cornell U.	ZPR	50-97	R-89	12-11-62	.1
	44. Ithaca	Cornell U.	TRIGA Mk. II	50-157	R-80	01-11-62	100
	45. Riverdale	Manhattan College	Tank	50-199	R-94	03-24-64	.0001
North Carolina	46. Raleigh	North Carolina State U.	PULSTAR	50-297	R-120	08-25-72	1000
Ohio	47. Columbus	Ohio State U.	Pool	50-150	R-75	02-24-61	10
Oklahoma	48. Norman	U. of Oklahoma	AGN-211 No. 102	50-112	R-53	12-29-58	.015
Oregon	49. Corvallis	Oregon State U.	AGN-201 No. 114	50-106	R-51	11-13-58	.0001
	50. Corvallis	Oregon State U.	TRIGA Mark II	50-243	R-106	03-07-67	1000
	51. Portland	Reed Institute	TRIGA Mk. I	50-288	R-112	07-02-68	250
Pennsylvania	52. University Park	Pennsylvania State U.	TRIGA Mk. III	50-5	R-2	07-08-55	1000

NON–POWER REACTORS IN THE U.S.

RESEARCH REACTORS – CONT.

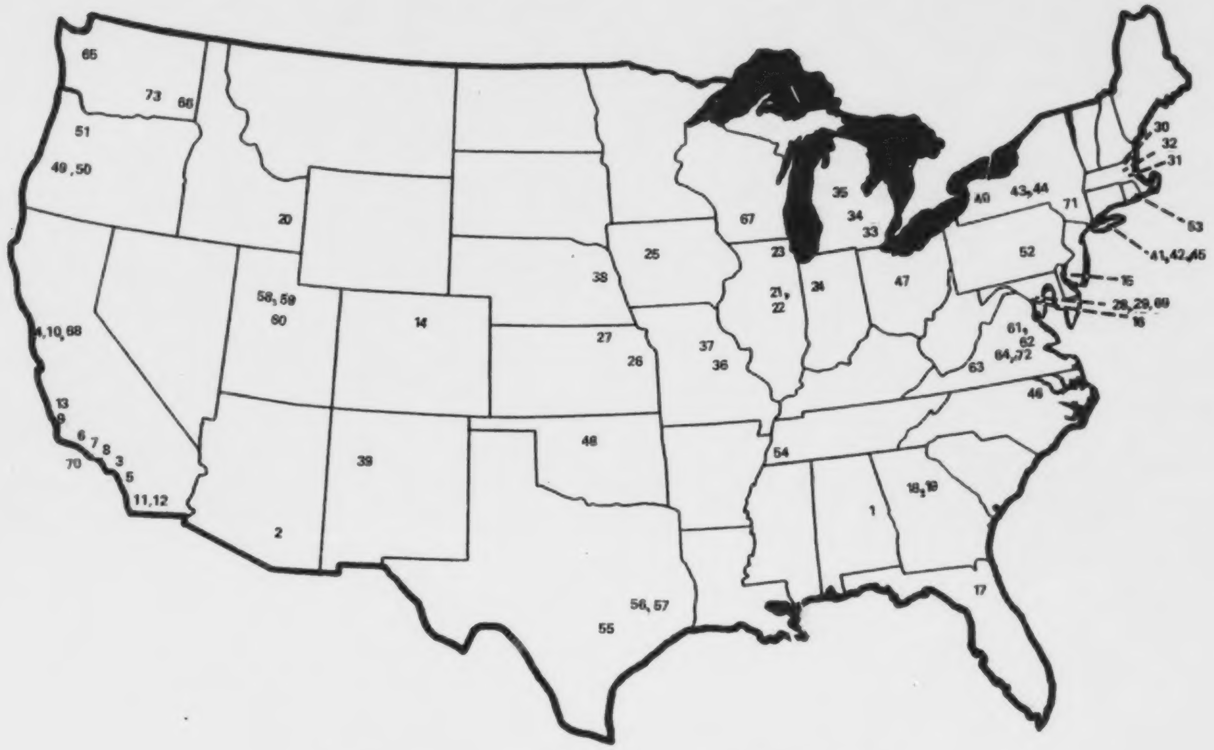
State	Location	Licensee	Reactor Type	Docket	License Number	Date Issued	Authorized Power Level (KW)
Rhode Island	53. Ft. Kearney, Narragansett	Rhode Island & Providence Plantations AEC	GE Pool	50-193	R-95	07-21-64	2000
Tennessee	54. Shelby County	Memphis State U.	AGN-201 No. 108	50-538	R-127	12-10-76	.0001
Texas	55. Austin	U. of Texas	TRIGA Mk. I	50-192	R-92	08-02-63	250
	56. College Station	Texas A&M U.	TRIGA	50-128	R-83	12-07-61	1000
	57. College Station	Texas A&M U.	AGN-201M No. 106	50-59	R-23	02-26-57	.005
Utah	58. Salt Lake City	U. of Utah	AGN-201M No. 107	50-72	R-25	09-12-57	.005
	59. Salt Lake City	U. of Utah	TRIGA Mark I	50-407	R-126	09-30-75	100
	60. Provo	Brigham Young U.	L-77	50-262	R-109	09-07-67	.01
Virgina	61. Charlottesville	U. of Virgina	Pool	50-62	R-66	06-27-60	2000
	62. Charlottesville	U. of Virginia	Cavalier	50-396	R-123	09-24-74	.1
	63. Blacksburg	Virginia Polytechnic Institute	UTR-10	50-124	R-62	12-18-59	100
	64. Lynchburg	Babcock & Wilcox Co.	LPR	50-99	R-47	09-05-58	1000
Washington	65. Seattle	U. of Washington	Argonaut	50-139	R-73	03-31-61	100
	66. Pullman	Washington State U.	TRIGA	50-27	R-76	03-06-61	1000
Wisconsin	67. Madison	U. of Wisconsin	TRIGA	50-156	R-74	11-23-60	1000

2. EXPERIMENTAL AND TEST REACTORS

State	Location	Licensee	Reactor Type	Docket	License Number	Date Issued	Authorized Power Level (MWt)
California	68. Alameda Co.	General Electric Co.	GETR	50-70	TR-1	01-07-59	50
Maryland	69. Gaithersburg	National Bureau of Standards	TEST	50-184	TR-5	11-13-67 POL 06-30-70 OL	10

3. CRITICAL EXPERIMENT FACILITIES

State	Location	Licensee	Docket	License Number	Date Issued	Experiments
California	70. Ventura Co.	Rockwell International Corp. (Possesion Only)	50-147	CX-17	10-03-60	AETR Experiments (Terminal Requested).
New York	71. Schenectady	Rensselaer Polytechnic Institute	50-225	CX-22	07-03-64	
Virginia	72. Lynchburg	Babcock & Wilcox Co.	50-13	CX-10	10-22-58	NMSR Experiments
Washington	73. Richland	Battelle Memorial Institute	50-360	CX-26	11-29-71	Plutonium Recycle Critical Facility



Source: NRC Facilities License Application Record (FLAR) 06-30-77.

ABBREVIATIONS

AGN	Aerojet General Nuclear	These reactors were constructed by Aeroject at San Ramon, California, and then transferred to the current location.
ZPR	Zero Power Reactor	
LPR	Low Power Reactor	

DISTRIBUTION LIST–NRC

INTERNAL DISTRIBUTION

Office of the Commissioners	5
Atomic Safety and Licensing Appeal Panel	3
Advisory Committee on Reactor Safeguards	6
Office of Inspector and Auditor	1
Office of Policy Evaluation	1
Office of the General Counsel	1
Office of Public Affairs	21
Office of Congressional Affairs	3
Office of the Executive Director for Operations	7
Office of Administration	2
Office of the Executive Legal Director	2
Office of the Controller	2
Office for Analysis and Evaluation of Operational Data	2
Office of International Programs	3
Office of State Programs	1
Office of Management and Program Analysis	14
Office of Standards Development	1
Office of Nuclear Material Safety and Safeguards	2
– Division of Fuel Cycle and Material Safety	3
– Division of Safeguards	2
Office of Nuclear Reactor Regulation Division of Licensing Division of Engineering Division of Systems Integration Division of Human Factors Safety Division of Safety Technology	138
Office of Nuclear Regulatory Research	4
Office of Inspection and Enforcement	7
– Region I	11
– Region II	10
– Region III	10
– Region IV	10
– Region V	7
	279

EXTERNAL DISTRIBUTION

SPECIAL REQUESTS

Air Force	1
Congress	10
Department of Energy	32
Government Accounting Office	1
Environmental Protection Agency	1
Electric Power Research Institute	1
Argonne National Laboratory	3
Federal Power Commission	1
Bureau of Mines	1
Department of Agriculture	1
Department of Commerce	1
Department of the Interior	1
	54

OTHER

GPO Depository	440
GPO Stores	75
National Technical Information Service	25
Subscriptions (NTIS)	250
Colleges and Libraries (including Public Document Rooms)	128
Utilities and Other Requests	227
	1145

15/4-9

Nuclear Regulatory Commission

NUREG 0020
VOL. 4 NO. 9
SEPTEMBER 1980

OPERATING UNITS STATUS REPORT

DATA AS OF 08/31/80

NEWARK PUBLIC LIBRARY
NOV 25 1980
U. S. [illegible]

LICENSED OPERATING REACTORS

Y3.N88:

Available from
GPO Sales Program
Division of Technical Information and Document Control
U.S. Nuclear Regulatory Commission
and
National Technical Information Service
Springfield, Virginia 22161
(703) 326-8543
A Year's Subscription, 12 Issues, for this Publication is
$120.00 North American Continent (240.00 Other).
Multiple Subscriptions Entered at the Same Time to the Same Address
Cost $90.00 North American Continent ($180.00 Other) for each Additional Subscription.

Single copies are $12.50 North American Continent ($25.00 Other).

This document is provided free of charge to those who participate in the data collection program, and to the Nuclear Regulatory Commission's local public document rooms.

Issued by the U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

PUBLISHED: OCTOBER 13, 1980

Nuclear Regulatory Commission

OPERATING UNITS STATUS REPORT

DATA AS OF 08/31/80

NUREG 0020
VOL. 4 NO. 9
SEPTEMBER 1980

LICENSED OPERATING REACTORS

AUTHORIZATION AND CLEARANCE

THE U.S. NUCLEAR REGULATORY COMMISSION'S OFFICE OF MANAGEMENT AND PROGRAM ANALYSIS PUBLISHES THIS MONTHLY STATUS REPORT "AS PART OF THE REPORTING REQUIREMENTS IN SECTION 50.36 OF 10 CFR PART 50 UNDER GAO CLEARANCE NUMBER B-180225, WITH AN EXPIRATION DATE OF SEPTEMBER 30, 1981," AS STATED IN THE OCTOBER 3, 1978 LETTER FROM JOHN M. LOVELADY, ASSISTANT DIRECTOR, GENERAL GOVERNMENT DIVISION, U. S. GENERAL ACCOUNTING OFFICE, TO J.M. FELTON, DIRECTOR, DIVISION OF RULES AND RECORDS, U.S. NUCLEAR REGULATORY COMMISSION.

STATEMENT OF PURPOSE

The U. S. Nuclear Regulatory Commission's monthly OPERATING UNITS STATUS REPORT — LICENSED OPERATING REACTORS, provides data on the operation of nuclear units as timely and accurately as possible. This information is collected by the Office of Management and Program Analysis from the Headquarters Staff of NRC's Office of Inspection and Enforcement, from NRC's Regional Offices, and from utilities. Since all of the data concerning operation of the units is provided by the utility operators less than two weeks after the end of the month, necessary corrections to published information are shown on the ERRATA page.

This report is divided into three sections: the first contains monthly highlights and statistics for commercial operating units, and errata from previously reported data; the second is a compilation of detailed information on each unit, provided by NRC Regional Offices, IE Headquarters and the Utilities; and the third section is an appendix for miscellaneous information such as spent fuel storage capability, reactor-years of experience and non-power reactors in the United States.

The percentage computations, Items 19 through 24 in Section 2, the vendor capacity factors on page 1-4, and actual vs. potential energy production on Page 1-2 are computed from actual power production from the period of consideration. The percentages listed in power generation on Page 1-2 are computed as an arithmetical average. The factors for the life-span of each unit (the "Cumulative" column) are reported by the utility and are not entirely re-computed by NRC. Utility power production data is checked for consistency with previously submitted statistics.

In some cases, the numbers in the "Cumulative" column may have been totaled from a date that is different from the date of commercial operation. In these cases, the column is noted with an explanation of the nature of the change on each individual plant page in Section 2.

It is hoped this status report proves informative and helpful to all agencies and individuals interested in analyzing trends in the nuclear industry which might have safety implications, or in maintaining an awareness of the U. S. energy situation as a whole.

TABLE OF CONTENTS

GLOSSARY ii
INDEX TO LICENSED UNITS iii

SECTION 1 – CURRENT DATA SUMMARIES

MONTHLY HIGHLIGHTS OF COMMERCIAL NUCLEAR POWER UNITS 1-2

- Licensed Power Reactors 1-2
- Power Generation 1-2
- Actual vs. Potential Energy Production 1-2
- Outage Data 1-2
- Derated Units 1-2
- Shutdowns - Number and Reasons 1-3
- Unit Availability, Capacity, and Forced Outage Rate 1-3
- Refueling Schedule 1-3

AVERAGE CAPACITY FACTORS BY VENDOR 1-4

AVERAGE DAILY POWER LEVEL FOR ALL COMMERCIAL OPERATING UNITS 1-5

ERRATA 1-7

SECTION 2 – OPERATING POWER REACTORS

ARKANSAS 1 THROUGH ZION 2 PAGES A–2 TO Z–5

- Facility Data
- Inspection and Enforcement Status
- Reports Received from Licensee
- Operating Status
- Average Daily Power Level (MWe)
- Unit Shutdowns/Reductions

SECTION 3 – APPENDIX

STATUS OF SPENT FUEL STORAGE CAPABILITY 3-2
REACTOR-YEARS OF EXPERIENCE 3-3
NON-POWER REACTORS IN THE U.S. 3-4

GLOSSARY

AVERAGE DAILY POWER LEVEL (MWe)	The net electrical energy generated during the day (measured from 0001 to 2400 hours inclusive) in megawatts hours divided by 24 hours.
LICENSED THERMAL POWER (MWt)	The maximum thermal power of the reactor authorized by the NRC, expressed in megawatts.
DATE OF COMMERCIAL OPERATION	Date unit was declared by utility owner to be available for the regular production of electricity; usually related to satisfactory completion of qualification tests as specified in the purchase contract and to accounting policies and practices of utility.
DESIGN ELECTRICAL RATING (DER) (Net MWe)	The nominal net electrical output of the unit specified by the utility and used for the purpose of plant design.
FORCED OUTAGE	An outage required to be initiated no later than the weekend following discovery of an off-normal condition.
FORCED OUTAGE HOURS	The clock hours during the report period that a unit is unavailable due to forced outages.
GROSS ELECTRICAL ENERGY GENERATED (MWH)	Electrical output of the unit during the report period as measured at the output terminals of the turbine generator, in megawatts hours.
GROSS HOURS	The clock hours from the beginning of a specified situation until its end. For outage durations, the clock hours during which the unit is not in power production.
GROSS THERMAL ENERGY GENERATED (MWH)	The thermal energy produced by the unit during the report period as measured or computed by the licensee in megawatt hours.
HOURS GENERATOR ON-LINE	Also, "Unit Service Hours." The total clock hours in the report period during which the unit operated with breakers closed to the station bus. These hours added to the total outage hours experienced by the unit during the report period, shall equal the hours in the report period.
HOURS IN REPORTING PERIOD	For units in power ascension at the end of the period, the gross hours from the beginning of the period or the first electrical production, whichever comes last, to the end of the period. For units in commercial operation at the end of the period, the gross hours from the beginning of the period or of commercial operation, whichever comes last, to the end of the period or decommissioning, whichever comes first.
HOURS REACTOR CRITICAL	The total clock hours in the report period during which the reactor sustained a controlled chain reaction.
MAXIMUM DEPENDABLE CAPACITY (GROSS) (MDC Gross) (Gross MWe)	Dependable main-unit gross capacity, winter or summer, whichever is smaller. The dependable capacity varies because the unit efficiency varies during the year due to cooling water temperature variations. It is the gross electrical output as measured at the output terminals of the turbine generator during the most restrictive seasonal conditions (usually summer).
MAXIMUM DEPENDABLE CAPACITY (NET) (MDC Net) (Net MWe)	Maximum Dependable Capacity (Gross) less the normal station service loads.
NAMEPLATE RATING (Gross MWe)	The nameplate power designation of the generator in megavolt amperes (MVA) times the nameplate power factor of the generator. NOTE: The nameplate rating of the generator may not be indicative of the maximum or dependable capacity, since some other item of equipment of a lesser rating (e.g. , turbine) may limit unit output.
NET ELECTRICAL ENERGY GENERATED	Gross electrical output of the unit measured at the output terminals of the turbine generator during the reporting period, minus the normal station service electrical energy utilization. If this quantity is less than zero, a negative number should be recorded.
OUTAGE	A situation in which no electrical production takes place.
OUTAGE DATE	As reported on Appendix D of Reg. Guide 1.16, the date of the start of the outage. If continued from a previous month, report the same outage date but change "Method of Shutting Down Reactor" to "4 (Other)" and add a note: "Continued from previous month."
OUTAGE DURATION	The total clock hours of the outage measured from the beginning of the report period or the outage, whichever comes last, to the end of the report period or the outage, whichever comes first.
OUTAGE NUMBER	A number unique to the outage assigned by the licensee. The same number is reported each month in which the outage is in progress. One format is "76-05" for the fifth outage to occur in 1976.
PERIOD HOURS	See "Hours in Reporting Period."
POWER REDUCTION	A reduction in the Average Daily Power Level of more than 20% from the previous day. All power reductions are defined as outage of zero hours durations for the purpose of computing unit service and availability factors, and forced outage rate.
REACTOR AVAILABLE HOURS	The total clock hours in the report period during which the reactor was critical or was capable of being made critical. (Reactor Reserve Shutdown Hours + Hours Reactor Critical.)
REACTOR AVAILABILITY FACTOR	$\frac{\text{Reactor Available Hours} \times 100}{\text{Period Hours}}$
REACTOR RESERVE SHUTDOWN	The cessation of criticality in the reactor for administrative or other similar reasons when operation could have been continued.
REACTOR RESERVE SHUTDOWN HOURS	The total clock hours in the report period during which the reactor is in reserve shutdown mode. NOTE: No credit is given for NRC imposed shutdowns.
REACTOR SERVICE FACTOR	$\frac{\text{Hours Reactor Critical} \times 100}{\text{Period Hours}}$
REPORT PERIOD	Usually, the preceding calendar month. Can also be the preceding calendar year, (Year-to-Date), or the life-span of a unit (cumulative).
RESTRICTED POWER LEVEL	Maximum net electrical generation to which the unit is restricted during the report period due to the state of equipment, external conditions, administrative reasons, or a direction by NRC.
SCHEDULED OUTAGE	Planned removal of a unit from service for refueling, inspection, training, or maintenance. Those outages which do not fit the definition of "Forced Outage" perforce are "Scheduled Outages."
STARTUP AND POWER ASCENSION TEST PHASE	Period following initial criticality during which the unit is tested at successively higher levels, culminating with operation at full power for a sustained period and completion of warranty runs. Following this phase, the utility generally considers the unit to be available for commercial operation.
UNIT	The set of equipment uniquely associated with the reactor, including turbine generators, and ancillary equipment, considered as a single electrical energy production facility.
UNIT AVAILABLE HOURS	The total clock hours in the report period during which the unit operated on-line or was capable of such operation. (Unit Reserve Shutdown Hours + Hours Generator On-Line.)
UNIT AVAILABILITY FACTOR*	$\frac{\text{Unit Available Hours} \times 100}{\text{Period Hours}}$
UNIT CAPACITY FACTORS	
– Using Licensed Thermal Power	$\frac{\text{Gross Thermal Energy Generated} \times 100}{\text{Period Hours} \times \text{Lic. Thermal Power}}$
– Using Nameplate Rating	$\frac{\text{Gross Electrical Energy Generated} \times 100}{\text{Period Hours} \times \text{Nameplate Rating}}$
– Using DER	$\frac{\text{Net Electrical Energy Generated} \times 100}{\text{Period Hours} \times \text{DER}}$
– Using MDC Gross	$\frac{\text{Gross Electrical Energy Generated} \times 100}{\text{Period Hours} \times \text{MDC Gross}}$
– Using MDC Net	$\frac{\text{Net Electrical Energy Generated} \times 100}{\text{Period Hours} \times \text{MDC Net}}$

NOTE: if MDC Gross and/or MDC Net have not been determined, the DER is substituted for this quantity for Unit Capacity Factor calculations.

UNIT FORCED OUTAGE RATE	$\frac{\text{Forced Outage Hours}}{\text{Unit Service Hours} + \text{Forced Outage Hours}}$
UNIT RESERVE SHUTDOWN	The removal of the unit from on-line operation for economic or other similar reasons when operation could have been continued.
UNIT RESERVE SHUTDOWN HOURS	The total clock hours in the report period during which the unit was in reserve shutdown mode.
UNIT SERVICE FACTOR	$\frac{\text{Unit Service Hours} \times 100}{\text{Period Hours}}$
UNIT SERVICE HOURS	See "Hours Generator On-Line."

INDEX TO LICENSED UNITS

ARKANSAS 1	A-3
ARKANSAS 2	A-5
BEAVER VALLEY 1	B-3
BIG ROCK POINT 1	B-5
BROWNS FERRY 1	B-7
BROWNS FERRY 2	B-9
BROWNS FERRY 3	B-11
BRUNSWICK 1	B-13
BRUNSWICK 2	B-15
CALVERT CLIFFS 1	C-3
CALVERT CLIFFS 2	C-5
COOK 1	C-7
COOK 2	C-9
COOPER STATION	C-11
CRYSTAL RIVER 3	C-13
DAVIS-BESSE 1	D-3
DRESDEN 1	D-5
DRESDEN 2	D-7
DRESDEN 3	D-9
DUANE ARNOLD	D-11
FARLEY 1	F-3
FITZPATRICK	F-5
FORT CALHOUN 1	F-7
FORT ST VRAIN	F-9
GINNA	G-3
HADDAM NECK	H-3
HATCH 1	H-5
HATCH 2	H-7
HUMBOLDT BAY	H-9
INDIAN POINT 2	I-3
INDIAN POINT 3	I-5
KEWAUNEE	K-3
LA CROSSE	L-3
MAINE YANKEE	M-3
MILLSTONE 1	M-5
MILLSTONE 2	M-7
MONTICELLO	M-9
NINE MILE POINT 1	N-3
NORTH ANNA 1	N-5
NORTH ANNA 2	N-7
OCONEE 1	O-3
OCONEE 2	O-5
OCONEE 3	O-7
OYSTER CREEK 1	O-9
PALISADES	P-3
PEACH BOTTOM 2	P-5
PEACH BOTTOM 3	P-7
PILGRIM 1	P-9
POINT BEACH 1	P-11
POINT BEACH 2	P-13
PRAIRIE ISLAND 1	P-15
PRAIRIE ISLAND 2	P-17
QUAD CITIES 1	Q-3
QUAD CITIES 2	Q-5
RANCHO SECO 1	R-3
ROBINSON 2	R-5
SALEM 1	S-3
SAN ONOFRE 1	S-5
ST LUCIE 1	S-7
SURRY 1	S-9
SURRY 2	S-11
THREE MILE ISLAND 1	T-3
THREE MILE ISLAND 2	T-5
TROJAN	T-7
TURKEY POINT 3	T-9
TURKEY POINT 4	T-11
VERMONT YANKEE 1	V-3
YANKEE-ROWE 1	Y-3
ZION 1	Z-3
ZION 2	Z-5

SECTION 1 – CURRENT DATA SUMMARIES

MONTHLY HIGHLIGHTS OF COMMERCIAL NUCLEAR POWER UNITS

FOR MONTH OF

AUGUST 1980

LICENSED POWER REACTORS:

NUMBER		CAPACITY[1] MWe (Net)
66	IN COMMERCIAL OPERATION	48,765
1	IN POWER ASCENSION PHASE	898
67[2]	LICENSED TO OPERATE	49,663
2[3]	LICENSED FOR FUEL LOADING AND LOW POWER TESTING	

POWER GENERATION	REPORT MONTH	PREVIOUS MONTH	YEAR-TO-DATE
1. GROSS ELECTRICAL (MWHe)	25,581,521	22,343,352	166,893,690
2. NET ELECTRICAL (MWHe)	24,312,485	21,165,600	158,818,744
3. AVG. UNIT SERVICE FACTOR (%)	77.8	65.8	63.2
4. AVERAGE UNIT AVAILABILITY FACTOR (%)	77.8	65.8	63.3
5. AVERAGE UNIT CAPACITY FACTOR (MDC) (%)	65.8	57.8	55.6
6. AVG. UNIT CAPACITY FACTOR (DER) (%)	64.0	56.3	54.1
7. FORCED OUTAGE RATE (%)	11.2	12.5	11.4

NOTES

[1] BASED UPON MAXIMUM DEPENDABLE CAPACITY; DESIGN ELECTRIC RATING USED IF MDC NOT DETERMINED.

[2] EXCLUDES DRESDEN 1 (DER = 200), HUMBOLDT BAY (DER = 65), AND THREE MILE ISLAND 2 (DER = 906) WHICH HAVE OPERATING LICENSES BUT ARE SHUTDOWN INDEFINITELY.

[3] SEQUOYAH 1 (DER = 1148) WAS LICENSED ON FEBRUARY 29, 1980
SALEM 2 (DER = 1115) WAS LICENSED ON APRIL 18, 1980

ACTUAL VS. POTENTIAL ENERGY PRODUCTION

			% OF POTENTIAL PRODUCTION
POTENTIAL ENERGY PRODUCTION IN THIS PERIOD BY UNITS IN COMMERCIAL OPERATION (Using Maximum Dependable Capacity Net)		36,281,160 MWHe	
ENERGY ACTUALLY PRODUCED DURING THIS REPORT PERIOD	=	24,312,485 MWHe NET	67.0
ENERGY NOT PRODUCED DURING THIS REPORT PERIOD DUE TO SCHEDULED OUTAGES (NET)	=	4,583,949 MWHe	12.6
ENERGY NOT PRODUCED DURING THIS REPORT PERIOD DUE TO FORCED OUTAGES (NET)	=	3,035,015 MWHe	8.4
ENERGY NOT PRODUCED DURING THIS REPORT PERIOD FOR OTHER REASONS (NET)	=	4,349,712 MWHe	12.0
TOTAL		36,281,160 MWHe =	100%
ENERGY NOT PRODUCED THIS REPORT PERIOD DUE TO NRC IMPOSED SHUTDOWNS AND RESTRICTIONS	=	1,796,167 MWHe	5.0
NUMBER OF UNITS WITH POWER RESTRICTIONS IMPOSED BY NRC	=	1	

OUTAGE DATA

	NUMBER	HOURS	PERCENT OF CLOCK TIME	MWHE LOST PRODUCTION
1. FORCED OUTAGES DURING MONTH	71	5,230.7	10.7	3,035,015
2. SCHEDULED OUTAGES DURING THE MONTH	27	5,671.9	11.6	4,583,949
3. TOTAL	98	10,902.6	22.2	7,618,963

MWHE LOST PRODUCTION = Down time x maximum dependable capacity net

DERATED UNITS

UNIT NAME	MAXIMUM DEPENDABLE CAPACITY (MWe - NET)	POWER LIMIT (MWe - NET)
COOPER STATION	764	635
† FORT ST VRAIN	330	231
INDIAN POINT 2	864	824
INDIAN POINT 3	965	917
POINT BEACH 1	495	390
ROBINSON 2	2300 MWT	2200 MWT

† Restricted by NRC.

MONTHLY HIGHLIGHTS OF COMMERCIAL NUCLEAR POWER UNITS

FOR MONTH OF

AUGUST 1980

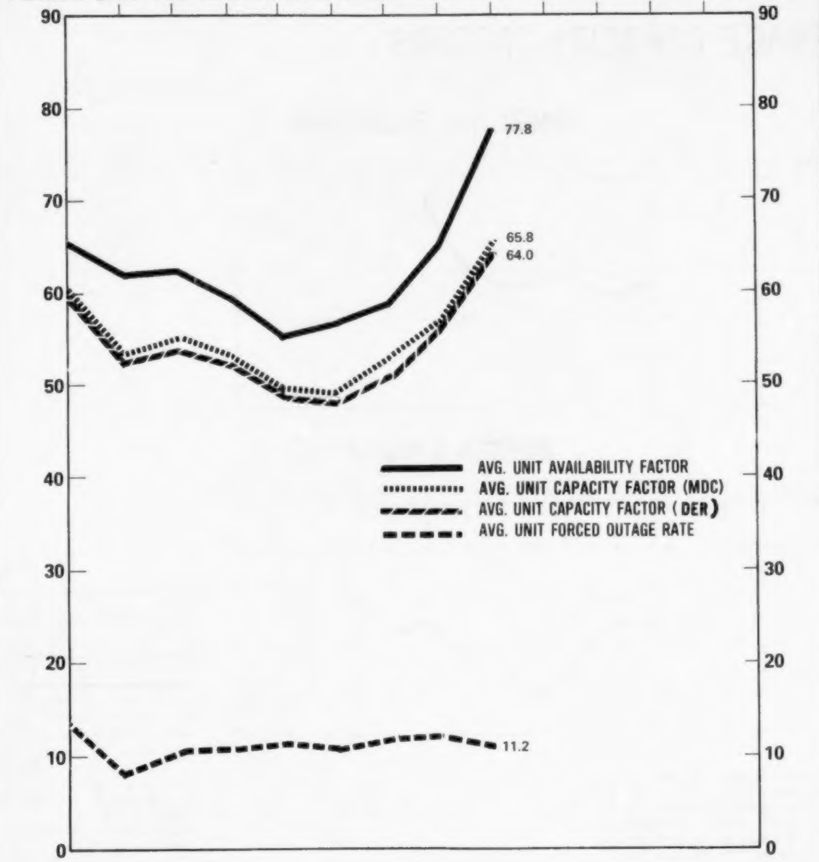
REASONS FOR SHUTDOWN

REASON	NUMBER	TOTAL HOURS LOST
A - Equipment Failure	45	3,367.2
B - Maintenance or Test	16	1,185.0
C - Refueling	11	3,887.8
D - NRC Restriction	6	2,207.4
E - Operator Training and License Examination	0	0
F - Administrative	2	22.1
G - Operational Error	13	142.6
H - Other	5	90.5
TOTAL	98	10,902.6

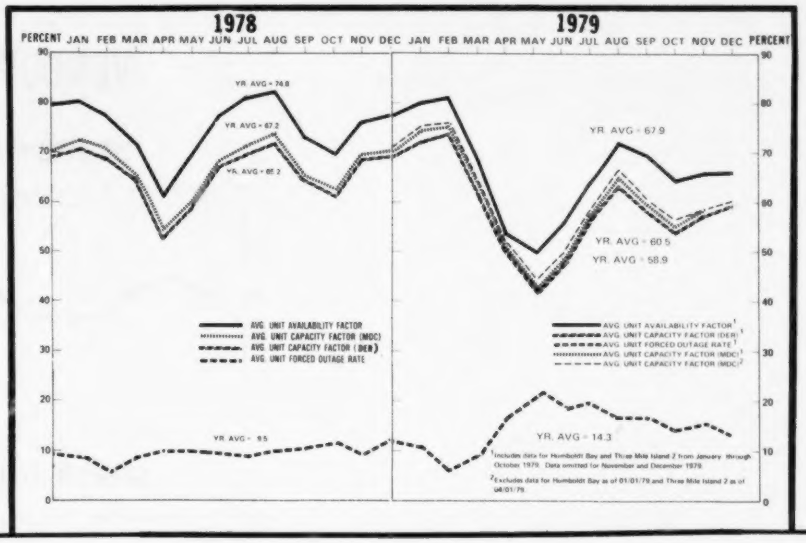
SHUTDOWNS GREATER THAN 72 HOURS EACH

UNIT NAME	REASON	UNIT NAME	REASON
ARKANSAS 1	A	OYSTER CREEK 1	B
BEAVER VALLEY 1	D	PALISADES	A
BRUNSWICK 1	C	PEACH BOTTOM 2	C
BRUNSWICK 2	C	PEACH BOTTOM 3	B
COOK 1	C	POINT BEACH 1	D
CRYSTAL RIVER 3	C	RANCHO SECO 1	A
DAVIS-BESSE 1	C	ROBINSON 2	C
FITZPATRICK	C	SAN ONOFRE 1	B
FORT ST VRAIN	A	SURRY 1	A
INDIAN POINT 2	A	SURRY 2	D
LA CROSSE	2A	THREE MILE ISLAND 1	D
MILLSTONE 2	C	YANKEE-ROWE 1	A
OCONEE 1	A		

UNIT AVAILABILITY, CAPACITY AND FORCED OUTAGE



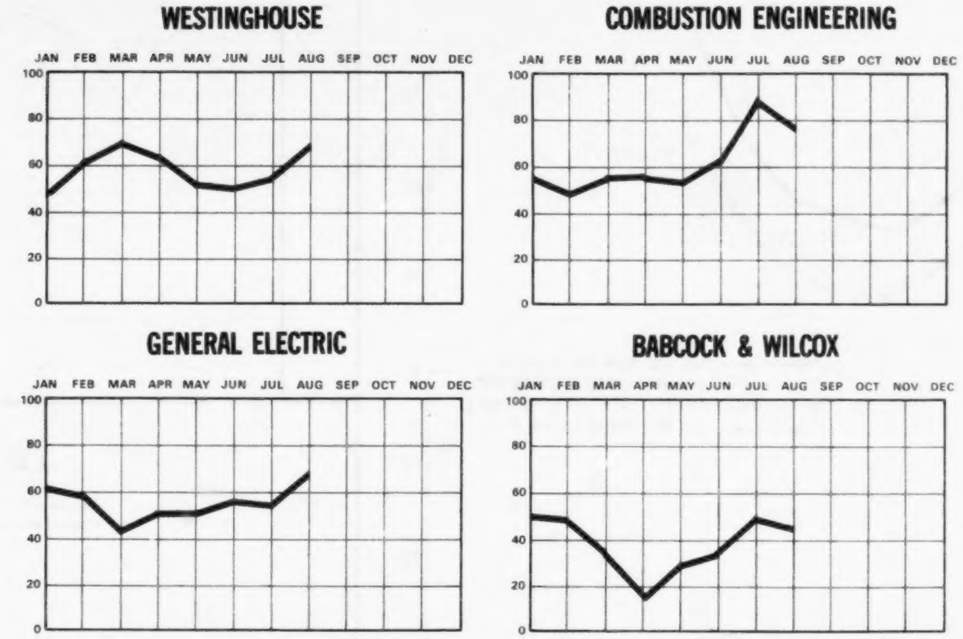
UNIT AVAILABILITY, CAPACITY AND FORCED OUTAGE



REFUELING SCHEDULE

Please refer to the table on page 3-2 for the scheduled refueling date for each nuclear powerplant in this publication.

VENDOR AVERAGE CAPACITY FACTORS



WESTINGHOUSE

COMBUSTION ENGINEERING

GENERAL ELECTRIC

BABCOCK & WILCOX

NOTE: THIS DISPLAY OF AVERAGE CAPACITY FACTORS PROVIDES A GENERAL PERFORMANCE COMPARISON OF PLANTS SUPPLIED BY THE FOUR NUCLEAR STEAM SUPPLY SYSTEM VENDORS. ONE MUST BE CAREFUL WHEN DRAWING CONCLUSIONS REGARDING THE REASONS FOR THE PERFORMANCE LEVELS INDICATED, SINCE PLANT PERFORMANCE MAY BE AFFECTED BY UNSPECIFIED FACTORS SUCH AS: (1) VARIOUS PLANT DESIGNS AND MODELS ARE INCLUDED FOR EACH VENDOR; (2) TURBINE/GENERATORS AND (3) DIFFERENT ARCHITECT/ENGINEERS ARE ALSO INVOLVED.

AUGUST 1980
OPERATIONS

GE BWR's	CFMDC
BROWNS FERRY 1	93.6
BROWNS FERRY 2	86.6
BROWNS FERRY 3	95.2
BRUNSWICK 1	9.4
BRUNSWICK 2	.0
COOPER STATION	69.5
DRESDEN 2	62.6
DRESDEN 3	85.2
DUANE ARNOLD	85.4
FITZPATRICK	27.7
HATCH 1	65.3
HATCH 2	93.8
MILLSTONE 1	97.5
MONTICELLO	96.2
NINE MILE POINT 1	90.8
OYSTER CREEK 1	63.5
PEACH BOTTOM 2	55.5
PEACH BOTTOM 3	71.9
PILGRIM 1	78.7
QUAD CITIES 1	53.9
QUAD CITIES 2	85.4
VERMONT YANKEE 1	89.6
22 UNITS	

W PWR's	
BEAVER VALLEY 1	.0
COOK 1	59.6
COOK 2	94.8
FARLEY 1	82.8
GINNA	98.4
HADDAM NECK	86.3
INDIAN POINT 2	72.6
INDIAN POINT 3	73.4
KEWAUNEE	91.6
NORTH ANNA 1	91.5
POINT BEACH 1	53.5
POINT BEACH 2	97.5
PRAIRIE ISLAND 1	63.3
PRAIRIE ISLAND 2	96.0
ROBINSON 2	9.5
SALEM 1	73.6
SAN ONOFRE 1	.0
SURRY 1	48.9
SURRY 2	13.4
TROJAN	92.8
TURKEY POINT 3	99.5
TURKEY POINT 4	98.6
YANKEE-ROWE 1	.0
ZION 1	77.9
ZION 2	75.8
25 UNITS	

CE PWR's	CFMDC
ARKANSAS 2	83.5
CALVERT CLIFFS 1	93.7
CALVERT CLIFFS 2	88.9
FORT CALHOUN 1	94.9
MAINE YANKEE	61.1
MILLSTONE 2	47.0
PALISADES	80.9
ST LUCIE 1	89.8
8 UNITS	

B&W PWR's	CFMDC
ARKANSAS 1	74.0
CRYSTAL RIVER 3	50.8
DAVIS-BESSE 1	.0
OCONEE 1	58.5
OCONEE 2	55.7
OCONEE 3	93.6
RANCHO SECO 1	34.8
THREE MILE ISLAND 1	.0
8 UNITS	

	GE BWR's	W PWR's	CE PWR's	B&W PWR's	All PWR's
NET ELECTRICAL PRODUCTION	8,917,829	9,402,891	3,571,048	2,311,555	15,285,494
MDC NET	17,154	18,340	6,090	6,737	31,167
CFMDC	69.9	68.9	78.3	46.1	65.9

Capacity factor in this page, denoted as CFMDC, is a function of the net maximum dependable capacity. See the corresponding definition in the Glossary. The vendor averages are computed by the formula:

$$\frac{\text{Net Electrical Energy Produced by Vendor}}{\text{Potential Electrical Production By Vendor In This Month}} \times 100\%$$

Units Excluded	Reason for Exclusion
Dresden 1 Humboldt Bay Three Mile Island 2	Unit Shutdown Indefinitely
Big Rock Point	Not Representative of Typical BWRs
LaCrosse	Not Representative of Typical BWRs and Atypical Vendor
Fort St. Vrain	Atypical Vendor and Reactor Design

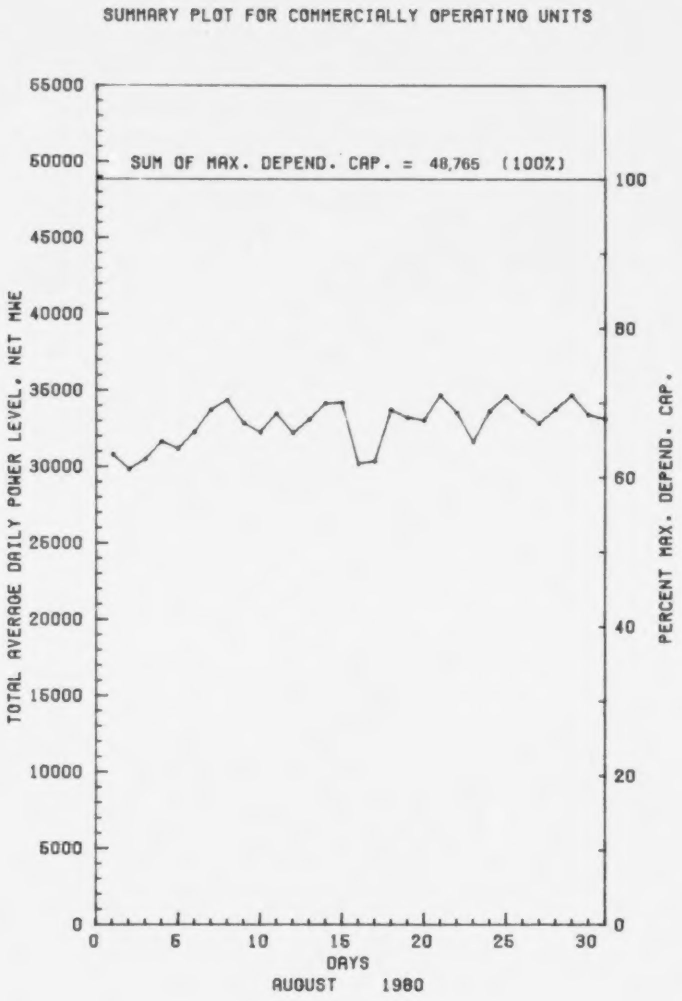
AVERAGE DAILY POWER LEVEL FOR ALL COMMERCIALLY OPERATING UNITS

This chart depicts the average daily power level for the units in commercial operation during the month.

The straight line on the graph labelled "SUM OF MAX. DEPEND. CAP." is plotted at the value shown by summing the separate maximum dependable capacities of the commercially operating units (in Net MWe). The plot shown below the line is calculated by summing the separate average daily power levels of the commercially operating units for each day of the month.

The scale on the left vertical axis runs in 1,000 MWe increments from 0 to 55,000 MWe (Net). The right vertical axis shows the percentage in 10% increments, up to 100% of the "SUM OF MAX. DEPEND. CAP."

It should be recognized that the 100% line would be obtainable only if all of the commercially operating units operated at 100% capacity, 24 hours per day, for the entire month. In other words, since any power generator must occasionally shut down to refuel and/or perform needed maintenance, and also since 100% capacity production is not always required by power demands, the 100% line is a theoretical goal and not a practical one.



ERRATA

Corrections and Amendments to Previously-Reported Data

(NOTE: These changes are reflected in the data contained in the current report)

CORRECTIONS TO NUREG-0020 VOL. 4, NO. 8 AUGUST 1980

Page	Unit Name	Item	Correction
R 5	ROBINSON 2	16. GROSS THERMAL ENERGY (MWH)	THE ENTRY FOR THE MONTH OF JULY SHOULD HAVE BEEN 870,283. YEAR TO DATE AND CUMULATIVE TOTALS SHOULD HAVE BEEN 8,014,423 AND 128,564,761 MWH, RESPECTIVELY.

MULTIPLE - ISSUE CORRECTIONS

NONE

SECTION 2 – OPERATING POWER REACTORS

ARKANSAS 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Arkansas/Pope 6 miles NW of Russellville
2.	Electric Reliability Council	Southwest Power Pool
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2568
5.	Design Electrical Rating (MWE Net)	850
6.	Date of Initial Criticality	August 6, 1974
7.	Date of Commercial Operation	December 19, 1974
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Dardenelle Reservoir

Utility & Contractor Information

10.	Licensee	Arkansas Power and Light Company
11.	Corporate Address	Ninth & Louisiana Streets Little Rock, Arkansas 72203
12.	Corporate Contact	W. Cavanaugh III Vice Pres., Generation & Construction
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	B&W
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	IV
17.	IE Branch Chief	G. L. Madsen
18.	IE Section Chief	D. M. Hunnicutt
19.	IE Resident Inspector	W. D. Johnson (501) 968-3290
20.	Licensing Project Manager	G. S. Vissing
21.	Docket Number	50-313
22.	License Number/Date of Issue	DPR-51, 5/21/74
23.	Public Document Room	Arkansas Polytechnic College Russellville, Arkansas 72801

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

Unit 1 shut down on July 16, 1980, to repair leaking steam generator tubes in "A" OTSG after the leak rate had increased to about 0.1 GPM. Three problem tubes were identified and plugged and Unit 1 was returned to power on August 5, 1980.

FACILITY ITEMS (PLANS AND PROCEDURES):

MANAGERIAL ITEMS

PLANT STATUS:

The Unit is operating at 90% power

LAST IE SITE INSPECTION DATE: June 22 - July 21, 1980

INSPECTION REPORT NO: 50-313/80-11

SUMMARY

INSPECTION SUMMARY

Inspection conducted during period of June 22 - July 21, 1980 (80-11): Routine, announced inspection including Surveillance Observation, Followup on IE Bulletins, Operational Safety Verification, Plant Trips, and Followup on Previously Identified Items. Within the five (5) areas inspected, one item of noncompliance was identified (infraction - Physical Security).

REPORTS RECEIVED FROM LICENSEE

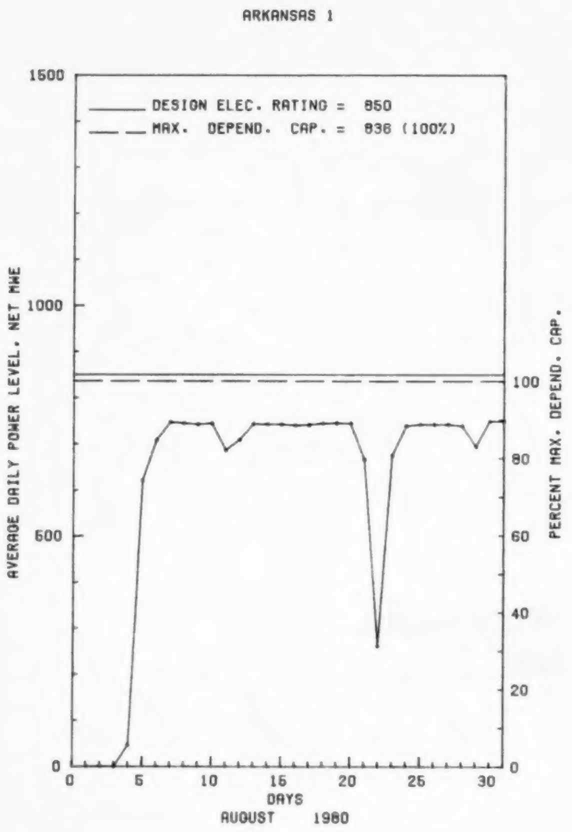
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-19 03L-0	6/19/80	7/18/80	Fire pump P6A inoperable
80-23 03L-0	7/9/80	8/4/80	Failure of P7-A steam supply valve
80-24 03L-0	7/6/80	8/4/80	Steam driven emergency feedwater pump trip
80-25 03L-0	7/11/80	8/7/80	Failure STBY H2 purge exhaust fan (Vef-37B)
80-26 03L-0	7/16/80	8/15/80	Steam generator tube leak
80-28 01T-0	7/31/80	8/12/80	Intake structure missile shield
80-27 04L-0	7/25/80	8/22/80	Average gross gas release rate
80-29 03L-0	7/23/80	8/22/80	Surveillance interval exceeded
80-30 99L-0	7/22/80	8/21/80	Cracked steam generator manway studs

REVIEWED BY OMPA LOEB: John L Crooks DATE: 10/6/80

REVIEWED BY NRR DL: DATE: 10/8/80

ARKANSAS 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-313
2. Utility Contact: L. S. Bramlett (501) 968-2519
3. Licensed Thermal Power (MWt): 2568
4. Nameplate Rating (Gross MWe): 1003 x 0.9 = 903
5. Design Electrical Rating (Net MWe): 850
6. Maximum Dependable Capacity (Gross MWe): 883
7. Maximum Dependable Capacity (Net MWe): 836
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	5,855.0	49,986.0
12. Hours Reactor Was Critical	655.1	3,277.3	33,415.5
13. Reactor Reserve Shutdown Hours	45.1	573.9	4,821.3
14. Hours Generator On-Line	646.1	3,221.4	32,691.9
15. Unit Reserve Shutdown Hours	.0	20.8	817.5
16. Gross Thermal Energy (MWH)	1,587,615	7,695,758	78,957,802
17. Gross Electrical Energy (MWH)	484,080	2,372,490	26,093,646
18. Net Electrical Energy (MWH)	460,420	2,257,060	24,891,096
19. Unit Service Factor	86.8	55.0	65.4
20. Unit Availability Factor	86.8	55.4	67.0
21. Unit Capacity Factor (MDC Net)	74.0	46.1	59.6
22. Unit Capacity Factor (DER Net)	72.8	45.4	58.6
23. Unit Forced Outage Rate	13.2	28.9	17.1

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
JANUARY 1, 1981 TO MARCH 1, 1981, REFUELING.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-07	07/16/80	F	88.3	A	1	80-026	CC	HTEXCH	A CONTINUATION OF JULY'S OUTAGE DUE TO THE "A" OTSG TUBE LEAK.
80-08	08/22/80	F	9.6	A	3		HH	MOTORX	A RUNBACK WAS INITIATED WHEN THE "B" MAINFEED PUMP TRIPPED DUE TO ITS OIL PUMP TRIPPING BECAUSE OF A MOTOR BEARING FAILURE. DURING THE RUNBACK THE RX. TRIPPED ON HIGH RCS PRESSURE WHEN "A" MAINFEED PUMP FAILED TO CONTROL PROPERLY. THE OIL PUMP MOTOR BEARING WAS REPLACED AND TRASH WAS CLEANED FROM THE FEED PUMP GOVERNOR MECHANISM.

SUMMARY

ARKANSAS 1 CAME ON LINE ON THE 5TH FROM AN OTSG TUBE LEAK AND OPERATED NORMALLY UNTIL THE 22ND WHEN AN OUTAGE OCCURRED DUE TO A MOTOR BEARING FAILURE.

LEGEND

[1] Type:
F-Forced
S-Scheduled

[2] Reason:
A-Equipment Failure
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error
H-Other

[3] Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Continuations
5-Load Reduction
9-Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

ARKANSAS 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Arkansas/Pope 6 miles NW of Russellville
2.	Electric Reliability Council	Southwest Power Pool
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2815
5.	Design Electrical Rating (MWE Net)	912
6.	Date of Initial Criticality	12-5-78
7.	Date of Commercial Operation	3/26/80
8.	Condenser Cooling Method	Recirculating
9.	Condenser Cooling Water Source	Cooling Tower

Utility & Contractor Information

10.	Licensee	Arkansas Power & Light Company
11.	Corporate Address	Ninth & Louisiana Streets Little Rock, Arkansas 72203
12.	Corporate Contact	W. Cavanaugh III Vice Pres., Generation & Construction
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Combustion Engineering, Inc.
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	IV
17.	IE Branch Chief	G. L. Madsen
18.	IE Section Chief	D. M. Hunnicutt
19.	IE Resident Inspector	W. D. Johnson (501) 968-3290
20.	Licensing Project Manager	R. E. Martin 492-8087
21.	Docket Number	50-368
22.	License Number/Date of Issue	NFP-6, 9/1/78
23.	Public Document Room	Arkansas Polytechnic College Russellville, Arkansas 72801

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO 10 CFR 20.203(B), THE 2T12 VALVE GALLERY ON ELEVATION 335' IN THE UNIT 2 AUX BUILDING WAS NOT POSTED AS A RADIATION AREA ALTHOUGH IN A PORTION OF THIS AREA, THE GENERAL RADIATION LEVEL WAS 20-30 MILLIREMS PER HOUR.
(8011 INFRACTION)

CONTRARY TO 10 CFR PART 20.203(C), "HIGH RADIATION AREAS," DOOR 364 WAS FOUND TO BE OPEN WITH NO POSITIVE ACCESS CONTROL. DOOR 364 PROVIDES ACCESS TO THE UNIT 2 F-15A AND B FILTER ROOM WHICH HAS A POSTED HIGH RADIATION AREA WITH ACCESSIBLE WHOLE-BODY RADIATION OF 100 MREM PER HOUR.
(8011 INFRACTION)

CONTRARY TO TECH SPEC 3.4.3, THE UNIT WAS OPERATED IN MODE 3 IN EXCESS OF 12 HOURS BEFORE THE OPERABILITY OF A PRESSURIZER CODE SAFETY VALVE WAS PROVED BY TESTING UNDER AMBIENT (HOT) CONDITIONS.
(8011 INFRACTION)

CONTRARY TO SECTION 10.3.5 OF FSAR, LICENSEE DOES NOT USE CATALYZED HYDRAZINE IN FEEDWATER AND DOES NOT SAMPLE AND ANALYZE FEEDWATER FOR AMMONIA AND HYDRAZINE FIVE TIMES PER WEEK.
(8011 DEVIATION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

FACILITY ITEMS (PLANS AND PROCEDURES):

MANAGERIAL ITEMS:

PLANT STATUS:

The Unit is operating at 100% power

LAST IE INSPECTION DATE: June 22 - July 21, 1980

INSPECTION REPORT NO: 50-368/80-11

SUMMARY

INSPECTION SUMMARY

Inspection Conducted During Period of June 22 - July 21, 1980 (80-11): Routine, announced inspection including Operation Safety Verification, Surveillance Observation, Maintenance, Followup on IE Bulletins, Pressurizer Code Safety Valve Testing, Plant Trips, and Followup on Previously Identified Items. Within the seven (7) areas inspected, four (4) items of noncompliance and one deviation were identified: (infraction - pressurizer code safety valves; infraction - posting of radiation area; infraction - access control to high radiation area; infraction - Physical Security; deviation - feedwater chemistry.

REPORTS RECEIVED FROM LICENSEE

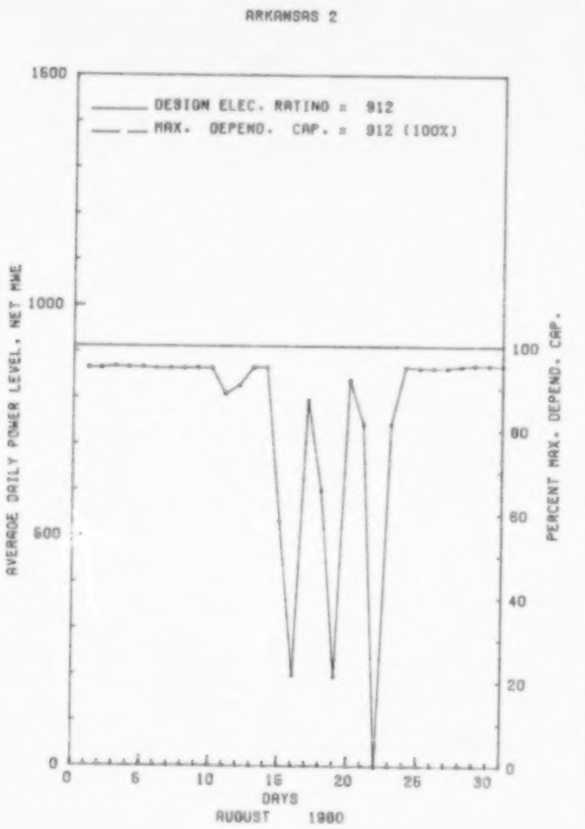
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
	NONE		

REVIEWED BY OMPA LOEB: MR Beebe **DATE:** 10/06/80

REVIEWED BY NRR DL: [illegible] **DATE:** [illegible]

ARKANSAS 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

UNIT NAME........................ ARKANSAS 2
FORCED OUTAGE HOURS.............. 52.3 715.1 715.1

1. Docket Number: 50-368
2. Utility Contact: Rex Pendergraft (501) 968-2519
3. Licensed Thermal Power (MWt): 2815
4. Nameplate Rating (Gross MWe): 959
5. Design Electrical Rating (Net MWe): 912
6. Maximum Dependable Capacity (Gross MWe): 912
7. Maximum Dependable Capacity (Net MWe): 912
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	3,815.0	3,815.0
12. Hours Reactor Was Critical	700.1	3,148.2	3,148.2
13. Reactor Reserve Shutdown Hours	43.9	124.8	124.8
14. Hours Generator On-Line	691.7	3,099.9	3,099.9
15. Unit Reserve Shutdown Hours	.0	75.0	75.0
16. Gross Thermal Energy (MWH)	1,845,897	7,773,645	7,773,645
17. Gross Electrical Energy (MWH)	593,560	2,523,800	2,523,800
18. Net Electrical Energy (MWH)	566,350	2,404,879	2,404,879
19. Unit Service Factor	93.0	81.3	81.3
20. Unit Availability Factor	93.0	83.2	83.2
21. Unit Capacity Factor (MDC Net)	83.5	69.1	69.1
22. Unit Capacity Factor (DER Net)	83.5	69.1	69.1
23. Unit Forced Outage Rate	7.0	18.7	18.7

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): MARCH 15, 1981 TO MAY 15, 1981, REFUELING.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-16	08/15/80	F	13.1	G	3		WG	ZZZZZZ	LOST STATOR COOLING WATER DUE TO HUMAN ERROR WHILE SWITCHING CONTROL ROOM CHILLERS AND MAIN CHILLERS.
80-17	08/16/80	F	5.0	A	3		CH	PIPEXX	LOW S/G LEVEL TRIP FROM APPROXIMATELY 17% POWER DUE TO FAULTY CONTROL OIL LINE FITTING. FITTING WAS REPLACED.
80-18	08/18/80	F	8.6	A	3		CH	INSTRU	LOW S/G LEVEL TRIP FROM FULL POWER DUE TO BLOWN FUSE IN FEEDWATER CONTROL SYSTEM CABINET. FUSE WAS REPLACED.
80-19	08/21/80	F	25.6	G	3		IA	INSTRU	HUMAN ERROR: DNBR/LPD TRIPS OCCURRED ON THE CPC'S DUE TO IMPROPER SWITCHING OF INVERTER POWER. (STATION BATTERY TO AC).

SUMMARY

ARKANSAS 2 OPERATED NORMALLY WITH THE EXCEPTION OF FOUR OUTAGES ON THE 15TH, 16TH, 18TH AND 21ST DUE TO EQUIPMENT FAILURE OR OPERATIONAL ERRORS.

LEGEND

[1] Type:
F-Forced
S-Scheduled

[2] Reason:
A-Equipment Failure
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error
H-Other

[3] Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Continuations
5-Load Reduction
9-Other

[4] Exhibit G:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

NRC POWER RESTRICTIONS: NONE

BEAVER VALLEY 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Pennsylvania/Beaver 5 miles E of E. Liverpool, Ohio
2.	Electric Reliability Council	East Central Area Reliability Coordination Agreement
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2660
5.	Design Electrical Rating (MWE Net)	852
6.	Date of Initial Criticality	May 10, 1976
7.	Date of Commercial Operation	10/01/76 (50%) 04/30/77 (100%)
8.	Condenser Cooling Method	Cooling tower/Closed loop
9.	Condenser Cooling Water Source	Ohio River

Utility & Contractor Information

10.	Licensee	Duquesne Light Company
11.	Corporate Address	435 Sixth Avenue Pittsburgh, Pennsylvania 15219
12.	Corporate Contact	C. N. Dunn, Vice President
13.	Architect/Engineer	Stone & Webster
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Section Chief	E. C. MCCABE
19.	IE Resident Inspector	D. A. BECKMAN (412-643-9200/9201)
20.	Licensing Project Manager	W. J. Ross
21.	Docket Number	50-334
22.	License Number/Date of Issue	DPR-66, 1/30/76
23.	Public Document Room	B. F. Jones Memorial Library 633 Franklin Avenue Aliquippa, PA 15001

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO 10 CFR 50, APPENDIX B, CRITERION V: BVPS FSAR, APPENDIX A.2, ATTACHMENT; ANSI N45.2.8, AND SAD 5 REQUIREMENTS FOR THE TEMPORARY USE OF SYSTEMS MODIFIED BY DCP 201/202 WERE NOT IMPLEMENTED DURING THE TURNOVER OF THE SYSTEM ON JANUARY 19, 1980 AND DURING SUBSEQUENT OPERATIONS THROUGH PLANT REFUELING.
(8009 INFRACTION)

CONTRARY TO 10 CFR 50, APPENDIX B, CRITERION VI: BVPS FSAR, APPENDIX A.2.2.6; QQA PROCEDURE OP-6, REVISION 0; AND QA PROCEDURE OP-4, REVISION 6, DOCUMENT CONTROL MEASURES WERE INADEQUATELY IMPLEMENTED IN THAT: 1) CONTROLLED INFORMATION WAS NOT MADE AVAILABLE TO PERSONNEL AS REQUIRED TO SUPPORT PREPARATION OF PROCEDURE REQUIRING REVISION AS A RESULT OF DCP 201/202; 2) STATION AND CONTROL ROOM CONTROLLED DRAWING FILES CONTAINED OBSOLETE DRAWINGS SUPERSEDED BY THOSE ISSUED AS A RESULT OF DCP 201/202; 3) AS-BUILT DRAWINGS ISSUED AS A RESULT OF DCP 201/202 WERE NOT TRANSMITTED TO THE STATION NOR ENTERED INTO STATION FILES PRIOR TO THEIR FIRST NEED FOR SAFETY RELATED ACTIVITIES; AND, 4) WHEN ISSUED FOR USE, AS-BUILT DRAWINGS ISSUED AS A RESULT OF DCP 201/202 HAD NOT RECEIVED FINAL ENGINEERING CHECKING.
(8009 INFRACTION)

CONTRARY TO 10 CFR 50, APPENDIX B, CRITERION XII: BVPS FSAR, APPENDIX A.2.2.12; AND, QA PROCEDURE OP-12, REVISION 3, SECTION 12.2.1, MEASURES FOR THE CONTROL AND CALIBRATION OF MEASUREMENT AND TEST EQUIPMENT WERE INADEQUATELY ESTABLISHED AND IMPLEMENTED; IN THAT THE BVPS MAINTENANCE MANUAL, CHAPTER 1, SECTION F, REVISION 10, DID NOT PROVIDE REQUIREMENTS FOR DIAL INDICATORS. AN UNCONTROLLED AND UNCALIBRATED DIAL INDICATOR WAS USED ON MARCH 24, 1980 DURING PERFORMANCE OF SAFETY-RELATED MAINTENANCE ON CONTROL ROOM AIR CONDITIONING COMPRESSOR NO. 40.
(8009 DEFICIENCY)

CONTRARY TO TS 6.8.1.C AND TS 3/4.3.3.1, SURVEILLANCE TEST PROCEDURE FOR RADIATION MONITORS RM-VS-103A AND B AND RM-VS-104A AND B WERE INADEQUATELY IMPLEMENTED AND MAINTAINED, IN THAT: 1) OST 1.43.1, REVISION 5 AND OST 1.44.C1, REVISION 1, PERFORMED, JANUARY - APRIL 1980 HAD NOT BEEN REVISED TO REFLECT THE ACTUAL SYSTEM CONFIGURATIONS RESULTING FROM RECENT MODIFICATIONS AND DID NOT TEST ALL REQUIRED FUNCTIONS; 2) OST 1.43.1, REVISION 5, PERFORMED ON MARCH 9, 1980 DOCUMENTED THE TESTS OF RM-103A AS SATISFACTORY WITHOUT NOTING EITHER PROCEDURE OR SYSTEM DEFICIENCIES EVEN THOUGH THE PROCEDURE DID NOT REFLECT ACTUAL SYSTEM CONFIGURATION AT THE TIME OF TEST; 3) OST 1.43.1, REVISION 8, WAS NOT ISSUED IN A TECHNICALLY CORRECT FORM PRIOR TO ACTIVITIES WHICH REQUIRED ITS PERFORMANCE; AND, 4) OST 1.43.1, REVISION 8, AS ORIGINALLY ISSUED ON MARCH 31, 1980 WAS TECHNICALLY INCORRECT IN THAT IT DID NOT TEST THE OPERATION OF ALL DAMPER/RADIATION MONITOR TRIP FUNCTIONS MODIFIED BY DCP 201/202.
(8009 INFRACTION)

SECTION 5.5.1 OF THE ENVIRONMENTAL TECHNICAL SPECIFICATIONS (ETS) REQUIRES THAT DETAILED WRITTEN PROCEDURES...WILL BE PREPARED AND IMPLEMENTED FOR ALL ACTIVITIES INVOLVING RELEASES TO THE ENVIRONMENT, DESCRIBED IN SECTIONS 2.0 AND 3.0, THE SAMPLING AND MONITORING OF THESE RELEASES AND THE CONDUCT OF THE ENVIRONMENTAL SURVEILLANCE PROGRAM. THESE PROCEDURES WILL INCLUDE SAMPLING, INSTRUMENT CALIBRATION...TESTING AND CALIBRATION FREQUENCIES OF EFFLUENT ALARMS AND MONITORING INSTRUMENTS. SECTION 3.1.1 OF THE (ETS) REQUIRES, IN PART, THAT A CONTINUOUS TEMPERATURE SENSOR WITH AN ACCURACY OF PLUS OR MINUS 0.5 DEGREES F OVER A RANGE OF 25-125 DEGREES F SHALL BE USED AT THE DISCHARGE STRUCTURE TO MEASURE COOLING TOWER BLOWDOWN. IF THE SENSORS LOCATED ON THE INLET SIDE OF THE CONDENSER SHALL BE USED AS A BACKUP. THE ACCURACY OF THESE SENSORS SHALL BE PLUS OR MINUS 1.0 DEGREES F OR LESS. CONTRARY TO THE ABOVE REQUIREMENTS, AS OF JUNE 6, 1980, WRITTEN PROCEDURES HAD NOT BEEN PREPARED NOR IMPLEMENTED FOR THE CALIBRATION OF THE DISCHARGE TEMPERATURE MONITORING SYSTEM RESISTANCE TEMPERATURE DETECTORS (RTDS) NOR FOR THE CALIBRATION OF THE BACKUP TEMPERATURE MONITORING SYSTEM CONDENSER INLET RTDS.
(8015 INFRACTION)

SECTION 5.5.1 OF THE ENVIRONMENTAL TECHNICAL SPECIFICATIONS (ETS) REQUIRES THAT DETAILED WRITTEN PROCEDURES...WILL BE PREPARED AND IMPLEMENTED FOR ALL ACTIVITIES INVOLVING RELEASES TO THE ENVIRONMENT, DESCRIBED IN SECTIONS 2.0 AND 3.0, THE SAMPLING AND MONITORING OF THESE RELEASES AND THE CONDUCT OF THE ENVIRONMENTAL SURVEILLANCE PROGRAM. THESE PROCEDURES WILL INCLUDE SAMPLING, INSTRUMENT CALIBRATION...TESTING AND CALIBRATION FREQUENCIES OF EFFLUENT ALARMS AND MONITORING INSTRUMENTS. SECTION 3.1.1 OF THE (ETS) REQUIRES, IN PART, THAT A CONTINUOUS TEMPERATURE SENSOR WITH AN ACCURACY OF PLUS OR MINUS 0.5 DEGREES F OVER A RANGE OF 25-125 DEGREES F SHALL BE USED AT THE DISCHARGE STRUCTURE TO MEASURE COOLING TOWER BLOWDOWN. IF THE

SEE CONTINUATION SHEET B-3A

REPORTS RECEIVED FROM LICENSEE

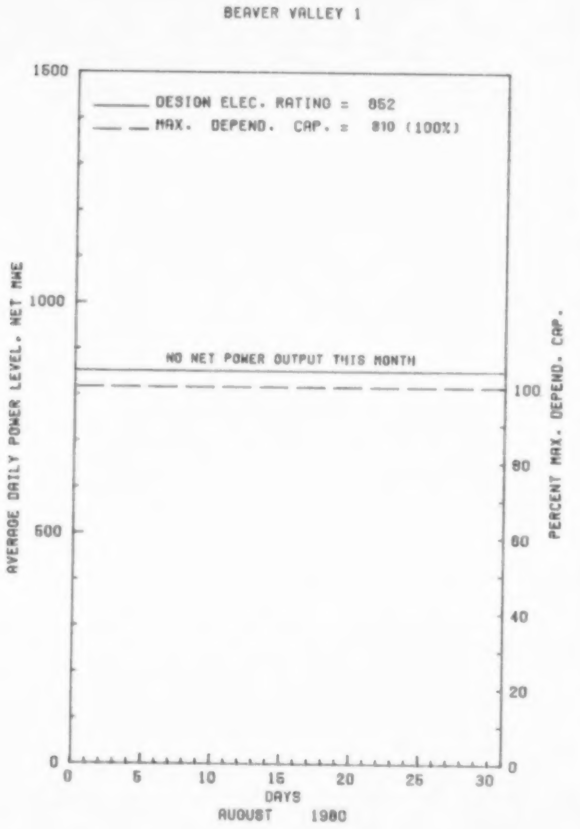
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-033/ 03L	06/17/80	07/15/80	GOVERNOR CONTROL MOTOR FOR EMERGENCY DIESEL GENERATOR NO. 1 FAILED WHEN OPERATORS ATTEMPTED TO SHUTDOWN EDG WITH FAST START SIGNAL STILL APPLIED. MOTOR REPLACED. OPERATOR COUNSELED.
80-043/ 03L	06/29/80	07/25/80	LOSS OF POWER TO BORIC ACID PUMP. DUE TO INCORRECT LABELING TEST ENGINEER TRIPPED CIRCUIT BREAKER TO PUMP DURING TEST. LICENSEE REVIEWING LABELING ON LOAD CENTERS.
80-046/ 03L	07/10/80	07/30/80	BOWED EMBEDMENT PLATE ON COMPONENT COOLING WATER LINE NOTED DURING IEB 79-14 INSPECTION. PROBABLE CAUSE IS WATER HAMMER PROCEDURES REVISED TO REQUIRE THROTTLED VALVE OPERATIONS.
80-048/ 03L	07/08/80	08/05/80	PIN HOLE LEAK DISCOVERED ON EXPANSION JOINT ON RIVER WATER DISCHARGE ON THE 1A RECIRCULATION SPRAY HEAT EXCHANGER. EXPANSION JOINT REPLACED PRIOR TO STARTUP.
80-049/ 01T	07/18/80	07/21/80	COMPONENT COOLING WATER SUPPLY LINE SUPPORT TO 1A RHR HEAT EXCHANGER DETERMINED TO BE SEISMICALLY INADEQUATE. SUPPORT WILL BE REINFORCED.
80-050/ 01T	07/18/80	07/21/80	AN UNDERSIZE SNUBBER HAD BEEN INSTALLED ON THE RECIRCULATION SPRAY LINE FROM THE 1A HEAT EXCHANGER TO ITS SPRAY HEADER. THE SNUBBER IS BEING REPLACED
80-051/ 01T	07/24/80	07/24/80	SEISMIC INADEQUACY OF THE B LOW HEAD SAFETY INJECTION PUMP RECIRCULATION LINE DETERMINED DURING IEB 79-14 REVIEW. NECESSARY SUPPORT MODIFICATIONS TO BE MADE PRIOR TO STARTUP.
80-052/ 01P	07/26/80	07/28/80	DIESEL FIRE PUMP AUTO-STARTED ON LOW SYSTEM PRESSURE. OBSERVED TO BE RUNNING BELOW RATED RPM AND PRESSURE. PUMP EXAMINED AND SUCCESSFULLY RERUN. BACKUP PUMP AVAILABLE.
80-053/ 01P	08/11/80	08/12/80	POTENTIAL OVERSTRESS CONDITION ON NORMAL CHARGING LINE PATHWAY DURING A SEISMIC EVENT. SUPPORTS TO BE REPLACED/MODIFIED PRIOR TO STARTUP.
80-054/ 01P	08/11/80	08/12/80	OVERSTRESS CONDITION ON SAFETY INJECTION LINE BETWEEN HI HEAD PUMPS AND RCS DURING A SEISMIC EVENT. SUPPORT TO BE REINFORCED PRIOR TO STARTUP.
80-055/ 01P	08/11/80	08/12/80	OVERSTRESS CONDITION ON REACTOR COOLANT LOOP TEMPERATURE MANIFOLDS (3) DURING SEISMIC EVENT. LINES TO BE REINFORCED PRIOR TO STARTUP.
80-056/ 01P	08/11/80	08/12/80	OVERSTRESS CONDITION ON SAFETY INJECTION LINE BETWEEN HI HEAD SAFETY INJECTION PUMPS AND THEIR RESPECTIVE RCS HOT LEGS DURING SEISMIC EVENT. SUPPORTS REINFORCED PRIOR TO STARTUP.
80-057/ 01T	08/19/80	08/20/80	POTENTIAL SEISMIC OVERSTRESS TO C STEAM GENERATOR BLOWDOWN LINE IN 3 LOCATIONS. SUPPORT MODIFICATIONS TO BE MADE PRIOR TO STARTUP. LINE LOSS WOULD LIMIT PLANT HEAT SINK CAPACITY.

REVIEWED BY OMPA LOEB: T. C. Cintula **DATE:** 10-6-80

REVIEWED BY NRR DL: W J Ross **DATE:** Oct 7, 1980

BEAVER VALLEY 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-334
2. Utility Contact: DAVID R. TIMKO (412) 643-5308
3. Licensed Thermal Power (MWt): 2660
4. Nameplate Rating (Gross MWe): 1026 x 0.9 = 923
5. Design Electrical Rating (Net MWe): 852
6. Maximum Dependable Capacity (Gross MWe): 845
7. Maximum Dependable Capacity (Net MWe): 810
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	5,855.0	38,015.0
12. Hours Reactor Was Critical	.0	.0	13,774.7
13. Reactor Reserve Shutdown Hours	.0	.0	4,482.7
14. Hours Generator On-Line	.0	.0	13,105.0
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	0	26,974,253
17. Gross Electrical Energy (MWH)	0	0	8,277,940
18. Net Electrical Energy (MWH)	0	0	7,570,743
19. Unit Service Factor	.0	.0	35.6
20. Unit Availability Factor	.0	.0	35.6
21. Unit Capacity Factor (MDC Net)	.0	.0	27.9
22. Unit Capacity Factor (DER Net)	.0	.0	26.5
23. Unit Forced Outage Rate	.0	.0	46.4

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 9/15/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
1	01/01/80	S	744.0	D	4		ZZ	ZZZZZZ	UNIT SHUTDOWN FOR MAJOR MODIFICATIONS AS REQUIRED BY THE NUCLEAR REGULATORY COMMISSION, INCLUDING NRC BULLETINS IEB 79-02 AND 79-14.

SUMMARY

BEAVER VALLEY 1 REMAINED SHUTDOWN FOR REQUIRED NRC SYSTEM MODIFICATIONS PER BULLETINS 79-02 AND 79-14.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

CONTINUATION PAGE

INSPECTION STATUS

SENSORS LOCATED ON THE INLET SIDE OF THE CONDENSER SHALL BE USED AS A BACKUP. THE ACCURACY OF THESE SENSORS SHALL BE PLUS OR MINUS 1.0 DEGREES F OR LESS. CONTRARY TO THE ABOVE REQUIREMENTS, AS OF JUNE 6, 1980, WRITTEN PROCEDURES HAD NOT BEEN PREPARED NOR IMPLEMENTED FOR THE CALIBRATION OF THE DISCHARGE TEMPERATURE MONITORING SYSTEM RESISTANCE TEMPERATURE DETECTORS (RTDS) NOR FOR THE CALIBRATION OF THE BACKUP TEMPERATURE MONITORING SYSTEM CONDENSER INLET RTDS.
(8015 INFRACTION)

CONTRARY TO 10 CFR 20.101, "RADIATION DOSE STANDARDS FOR INDIVIDUALS IN RESTRICTED AREAS", DURING THE FIRST CALENDAR QUARTER OF 1980, AN INDIVIDUAL WAS PERMITTED TO RECEIVE IN EXCESS OF 1.25 REM. THE PRESCRIBED FORM NRC-4 HAD NOT BEEN COMPLETED.
(8022 INFRACTION)

10 CFR 50, APPENDIX B, CRITERION IX, STATES, IN PART, THAT "...SPECIAL PROCESSES, INCLUDING...WELDING ARE CONTROLLED AND ACCOMPLISHED...IN ACCORDANCE WITH APPLICABLE CODES, STANDARDS, SPECIFICATIONS...." THE BEAVER VALLEY POWER STATION PSAR, SECTION 17, PARAGRAPH 17.2.1.9A, ALSO STATES, IN PART, THAT: "...THE DLC QAP ESTABLISHES MEASURES TO ASSURE THAT SPECIAL PROCESSES, INCLUDING WELDING...ARE CONTROLLED IN ACCORDANCE WITH APPLICABLE CODES, STANDARDS, SPECIFICATIONS...." DLC QA MANUAL IN QA PROCEDURE DC-9 (CONTROL OF SPECIAL PROCESSES), PARAGRAPH 9.4.2 STATES IN PART THAT "THE AE/CONSTRUCTOR AND NSS SUPPLIER ARE RESPONSIBLE FOR THE ESTABLISHMENT AND IMPLEMENTATION OF MEASURES FOR CONTROLLING SPECIAL PROCESSES...AND SHALL SUPPLY CONTRACTORS WITH PROCEDURES FOR CONTROLLING SPECIAL PROCESSES." STONE AND WEBSTER FIELD CONSTRUCTION PROCEDURE, FCP-601.5, PARAGRAPH 11.12, STATES, IN PART, "EACH WELDER SHALL BE RESPONSIBLE FOR ALL WELDING MATERIALS.... ALL ELECTRODE STUBS OR DAMAGED ELECTRODES SHALL BE RETAINED BY THE WELDER IN A SUITABLE CONTAINER (STUB BUCKET) FOR PROPER DISPOSAL AT THE ISSUE STATION." CONTRARY TO THE ABOVE, ON JUNE 3, 1980, APPROXIMATELY 25-40 WELDING ELECTRODE STUBS AND DAMAGED ELECTRODES HAD BEEN PLACED IN A LARGE TRASH CONTAINER WITHIN THE CONTAINMENT BUILDING. THIS ITEM IS AN INFRACTION.
(8006 INFRACTION)

10 CFR 50, APPENDIX B, CRITERION XIII, STATES, IN PART, "MEASURES SHALL BE ESTABLISHED TO CONTROL THE HANDLING, STORAGE, SHIPPING, CLEANING AND PRESERVATION OF MATERIAL AND EQUIPMENT...TO PREVENT DAMAGE OR DETERIORATION." THE BEAVER VALLEY POWER STATION UNIT 2, PSAR SECTION 17, PARAGRAPH 17.2.1.13A ALSO STATES, "THE DLC QAP ESTABLISHES MEASURES TO CONTROL THE HANDLING, STORAGE, SHIPPING, CLEANING AND PRESERVATION OF MATERIAL AND EQUIPMENT IN ACCORDANCE WITH WORK AND INSPECTION INSTRUCTIONS TO PREVENT DAMAGE OR DETERIORATION." DLC QA MANUAL IN QA PROCEDURE DC-13 (HANDLING, STORAGE AND SHIPPING), PARAGRAPH 13.3 STATES IN PART THAT "SPECIFIC PROCEDURES AND INSTRUCTIONS SHALL BE WRITTEN WHEN NECESSARY FOR THE PRESERVATION, PACKAGING AND STORAGE OF ITEMS. THE PROCEDURES AND INSTRUCTIONS SHALL BE UTILIZED TO ASSURE THAT ITEMS SUBJECTED TO DETERIORATION OR DAMAGE...ARE CLEANED AND HAVE NECESSARY PRESERVATIVES TO PREVENT DAMAGE." STONE AND WEBSTER FCP-11 ATTACHMENT 3.2 TO APPENDIX 1 INDIC ATES THAT VALVES ARE CLASSIFIED FOR STORAGE UNDER STORAGE LEVEL C. THE DOCUMENT STATES THAT FOR L EVEL C STORAGE "ITEMS SHALL BE STORED INDOORS OR EQUIVALENT WITH ALL PROVISIONS AND REQUIREMENTS A S STATED FOR LEVEL B ITEMS EXCEPT THAT HEAT AND TEMPERATURE CONTROLS ARE NOT REQUIRED." LEVEL B STATES "THE ITEMS SHALL BE STORED WITHIN A FIRE RESISTANT, TEAR RESISTANT, WEATHER TIGHT AND WELL VENTILATED BUILDING OR EQUIVALENT ENCLOSURE...." CONTRARY TO THE ABOVE, ON JUNE 3, 1980, THE VALVE 2-VV1-015-11-3, SERIAL NO. D-0021-3-2, WAS STORED IN AN OPEN BOX AT THE BOTTOM LEVEL OF THE CONTAINMENT BUILDING, EXPOSED TO THE WEATHER AND AS A RESULT SHOWED EVIDENCE OF WET CORROSION. THIS ITEM IS AN INFRACTION.
(8006 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

FACILITY ITEMS (PLANS AND PROCEDURES):

- Refueling/modification outage commenced 11/30/79; outage is expected to continue until at least 9/15/80.

PLANT STATUS:

- IN COLD SHUTDOWN. WORK CONTINUES ON BERGEN-PATTERSON LARGE BORE SNUBBERS, REFUELING WATER STORAGE TANK, QUENCH SPRAY SYSTEMS, PIPEHANGERS, SUPPORTS, BASE PLATES, LOW HEAD SI PUMPS, AND THE REACTOR COOLANT PUMP MOTOR THRUST RUNNER MODIFICATIONS. LOW PRESSURE TURBINE ROTOR INSTALLATION IS COMPLETED.

- LAST IE SITE INSPECTION DATE: 8/25-29/80

- INSPECTION REPORT NO: 50-334/80-24

SUMMARY

INSPECTION SUMMARY

- 50-334/80-15 - JUN 2-6: ROUTINE, UNANNOUNCED INSPECTION BY ONE REGION-BASED INSPECTOR (40 HRS) OF ENVIRONMENTAL MONITORING PROGRAMS FOR OPERATIONS INCLUDING: THE MANAGEMENT CONTROLS FOR THESE PROGRAMS; THE LICENSEE'S PROGRAM FOR QUALITY CONTROL OF ANALYTICAL MEASUREMENTS; IMPLEMENTATION OF THE ENVIRONMENTAL MONITORING PROGRAMS - RADIOLOGICAL; BIOLOGICAL/ECOLOGICAL; AND NONRADIOACTIVE EFFLUENT RELEASE RATES AND LIMITS. ONE ITEM OF NONCOMPLIANCE WAS IDENTIFIED: FAILURE TO HAVE COMPLETE DISCHARGE TEMPERATURE MONITORING SYSTEM CALIBRATION PROCEDURE.

- 50-334/80-21 - JUL 18: SPECIAL MANAGEMENT MEETING ATTENDED BY SENIOR LICENSEE AND REGIONAL MANAGEMENT TO DISCUSS NRC CONCERNS REGARDING LICENSEE IMPLEMENTATION OF ADMINISTRATIVE CONTROLS, LICENSEE ACTION TAKEN IN RESPONSE TO RECENT INCIDENTS INVOLVING OUTAGE RECOVERY ACTIVITIES AND PLANT STAFF WORK HOURS.

- 50-334 - JUL 28 - AUG 1: ROUTINE, UNANNOUNCED INSPECTION BY A REGION-BASED INSPECTOR (38 HRS) OF THE RADIATION PROTECTION PROGRAM DURING THE REFUELING OUTAGE INCLUDING: ADVANCED PLANNING AND PREPARATION; TRAINING; EXPOSURE CONTROL; RESPIRATORY PROTECTION PROGRAM; POSTING, LABELING, AND CONTROL; SURVEYS; AND PLANT TOURS. LICENSEE ACTIONS ON PREVIOUS INSPECTION FINDINGS WERE REVIEWED, AND AREAS WHERE WORK WAS BEING CONDUCTED WERE EXAMINED TO REVIEW RADIOLOGICAL PROCEDURES AND PRACTICES. ONE ITEM OF NONCOMPLIANCE WAS IDENTIFIED: FAILURE TO COMPLETE FORM NRC-4 PRIOR TO EXCEEDING 1.25 REM/QTR.

BIG ROCK POINT 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Michigan/Charlevoix 4 miles NE of Charlevoix
2.	Electric Reliability Council	East Central Area Reliability Coordination Agreement
3.	Type of Reactor	BWR
4.	Power Level (MWT)	240
5.	Design Electrical Rating (MWE Net)	72
6.	Date of Initial Criticality	September 27, 1962
7.	Date of Commercial Operation	March 29, 1963
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10.	Licensee	Consumers Power Company
11.	Corporate Address	212 West Michigan Avenue Jackson, Michigan 49201
12.	Corporate Contact	R. B. DeWitt
13.	Architect/Engineer	Bechtel Corporation
14.	Nuclear Steam Supply System	GE
15.	Constructor	Bechtel Corporation

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	D. C. BOYD
19.	IE Resident Inspector	G. Wright
20.	Licensing Project Manager	W. A. Paulson
21.	Docket Number	50-155
22.	License Number/Date of Issue	DPR-6 issued 8/30/62
23.	Public Document Room	Charlevoix Public Library 107 Clinton Street Charlevoix, Michigan

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

Operating routinely.

LAST IE SITE INSPECTION DATE: July 3 through August 4, 1980

INSPECTION REPORT NO: 80-09

SUMMARY

INSPECTION SUMMARY

Inspection on June 11 and 26, (80-08): Confirmatory Measurements including: telephone discussion and closeout of results of a previous sample collection. The inspection involved 0 inspector hours onsite by one NRC inspector. No items of noncompliance or deviations were identified.

Inspection on July 3 through August 4, (80-09): Routine Resident Inspector inspection of operational safety, bulletin review and activities pertaining to Bulletin 80-17. The inspection involved 93 inspector hours onsite including 20 hours off-shift inspection by one NRC inspector. No apparent items of noncompliance were identified in the areas inspected.

REPORTS RECEIVED FROM LICENSEE

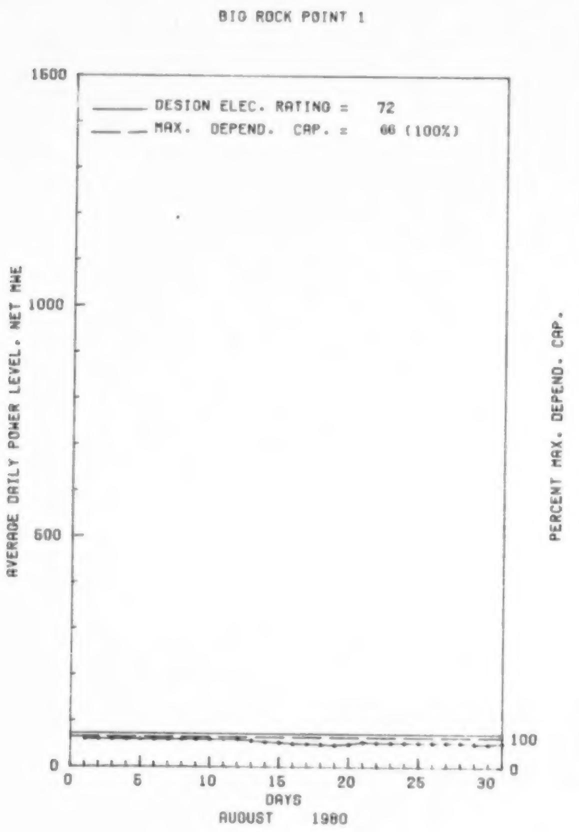
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-17/ 03L-0	07/03/80	08/13/80	Sensor Channel D of the Reactor Depressurization System was removed from service to correct the intermittent automatic test circuit alarm.
80-18/ 03L-0	07/10/80	08/04/80	During monthly specific gravity checks Cell 35 and 54 of the battery power supply for Channel C of the Reactor Depressurization System read 1.192 and 1.194, respectively.
80-19/ 03L-0	07/13/80	08/12/80	A visual alarm was noted on a containment pressure indicator on Unit PIS 173 in the control room.
80-20/ 03L-0	07/18/80	08/15/80	One 3 unit cell was defective in the Reactor Depressurization System battery for Channel C.

REVIEWED BY OMPA LOEB: John C. Crooks DATE: 10/6/80

REVIEWED BY NRR DL: Walter A. Paulson DATE: 10/8/80

BIG ROCK POINT 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

ITEM 21 CALCULATED WITH A WEIGHTED AVERAGE

1. Docket Number: 50-155
2. Utility Contact: G. Himebauch (616) 547-6537 x180
3. Licensed Thermal Power (MWt): 240
4. Nameplate Rating (Gross MWe): 70.6 x 0.85 = 60
5. Design Electrical Rating (Net MWe): 72
6. Maximum Dependable Capacity (Gross MWe): 70
7. Maximum Dependable Capacity (Net MWe): 65
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): 65
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	5,855.0	152,778.0
12. Hours Reactor Was Critical	744.0	5,495.6	105,628.4
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	744.0	5,468.5	103,467.4
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	142,602	1,127,391	19,245,322
17. Gross Electrical Energy (MWH)	43,839	344,318	6,080,236
18. Net Electrical Energy (MWH)	41,396	325,306	5,753,643
19. Unit Service Factor	100.0	93.4	67.7
20. Unit Availability Factor	100.0	93.4	67.7
*21. Unit Capacity Factor (MDC Net)	84.3	84.2	55.6
22. Unit Capacity Factor (DER Net)	77.3	77.2	52.3
23. Unit Forced Outage Rate	.0	.7	23.0

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUELING OUTAGE SCHEDULED TO BEGIN ON OCTOBER 31, 1980.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	NONE								

SUMMARY

BIG ROCK POINT OPERATED NORMALLY FOR THE ENTIRE MONTH WITH NO OUTAGES REPORTED.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

BROWNS FERRY 1

FACILITY DATA

Facility Description

1. Location (state/county) (nearest population center)	Alabama/Limestone 10 miles NW of Decatur
2. Electric Reliability Council	Southeastern Electric Reliability Council
3. Type of Reactor	BWR
4. Power Level (MWT)	3293
5. Design Electrical Rating (MWE Net)	1065
6. Date of Initial Criticality	August 17, 1973
7. Date of Commercial Operation	August 1, 1974
8. Condenser Cooling Method	Once Through With Forced Draft Cooling Towers
9. Condenser Cooling Water Source	Tennessee River

Utility & Contractor Information

10. Licensee	Tennessee Valley Authority
11. Corporate Address	500A Chestnut Street Tower II Chattanooga, Tennessee 37401
12. Corporate Contact	H. G. Parris, Manager of Power.
13. Architect/Engineer	TVA
14. Nuclear Steam Supply System	GE
15. Constructor	TVA

Regulatory Information

16. IE Region responsible	II
17. IE Branch Chief	R. C. Lewis
18. IE Section Chief	H. C. Dance
19. IE Resident Inspector	R. Sullivan (205) 729-6196
20. Licensing Project Manager	R. Clark
21. Docket Number	50-259
22. License Number/Date of Issue	DPR-33, 6/26/73
23. Public Document Room	Athens Public Library South and Forrest Athens, Alabama 35611

INSPECTION STATUS

ENFORCEMENT STATUS

AS REQUIRED BY TECHNICAL SPECIFICATION 6.3.A, DETAILED WRITTEN PROCEDURES SHALL BE PREPARED, APPROVED, AND ADHERED TO FOR SURVEILLANCE AND TESTING REQUIREMENTS. CONTRARY TO THE ABOVE, ON MAY 6, 1980, UNITS 1, 2 AND 3 CONTROL ROOM OPERATORS WERE PERFORMING SURVEILLANCE INSTRUCTION 2, INSTRUMENT CHECKS AND OBSERVATIONS, USING A SECTION WHICH HAD BEEN SUPERCEDED BY AN APPROVED REVISION DATED JULY 10, 1979. (8024 DEFICIENCY)

AS REQUIRED BY 10 CFR 50, APPENDIX B, AND IMPLEMENTED BY TENNESSEE VALLEY AUTHORITY NUCLEAR - OPERATIONAL QUALITY ASSURANCE MANUAL SECTION 2.1 PART II, UPON COMPLETION OF MAINTENANCE ON ANY ITEM OF THE CSSC LIST AND BEFORE RELEASE FOR SERVICE, APPROPRIATE TESTING SHALL BE PERFORMED TO VERIFY OPERATIONAL ACCEPTABILITY. CONTRARY TO THE ABOVE, ON JUNE 3, 1980, THE CONTROL VALVE 12 PRESSURE SENSOR, WHICH SENDS A SIGNAL TO THE REACTOR PROTECTION SYSTEM TO INDICATE TURBINE CONTROL VALVE FAST CLOSURE, HAD MAINTENANCE PERFORMED ON IT AND WAS RETURNED TO SERVICE WITHOUT TESTING OF THE REACTOR PROTECTION SYSTEM TO VERIFY PROPER OPERATION OF THE REPAIRED PRESSURE SWITCH. (8026 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

100 PERCENT FULL POWER OPERATION.

LAST IE SITE INSPECTION DATE: JUNE 28 - JULY 4, 1980

INSPECTION REPORT NO: 50-259/80-32

SUMMARY

INSPECTION SUMMARY

INSPECTION JUNE 1-30 (80-28): THIS ROUTINE INSPECTION INVOLVED 37 RESIDENT INSPECTOR-HOURS IN THE AREAS OF OPERATIONAL SAFETY MAINTENANCE, NONCOMPLIANCE FOLLOWUP, REPORTABLE OCCURRENCES, REACTOR TRIP FOLLOWUP, PLANT PHYSICAL PROTECTION AND RADIATION SAFETY. OF THE SEVEN AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED IN SIX AREAS. ONE ITEM OF NONCOMPLIANCE WAS FOUND IN ONE AREA (INFRACTION - FAILURE TO TEST A PORTION OF THE REACTOR PROTECTION SYSTEM AFTER MAINTENANCE).

INSPECTION JULY 7-10 (80-29): THE INSPECTION INVOLVED 10 INSPECTOR HOURS ON SITE BY ONE NRC INSPECTOR. THREE INSPECTION HOURS WERE ACCOMPLISHED DURING OFFSHIFT PERIODS. AREAS INSPECTED INCLUDED REVIEW OF SECURITY PLAN; SECURITY ORGANIZATION; PHYSICAL BARRIERS (VITAL AND PROTECTED AREAS); LIGHTING; ASSESSMENT AIDS; ACCESS CONTROLS (IDENTIFICATION, AUTHORIZATION, AND BADGING, SEARCH, ESCORTING AND VITAL AREAS); AND ALARM STATION. THE LICENSEE WAS FOUND TO BE IN COMPLIANCE WITH NRC REQUIREMENTS WITHIN 11 AREAS EXAMINED DURING THE INSPECTION.

INSPECTION JUNE 28 - JULY 4 (80-32): THIS SPECIAL, ANNOUNCED INSPECTION INVOLVED APPROXIMATELY 100 INSPECTOR-HOURS ON SITE IN THE AREAS OF INSPECTION AND EVALUATION OF THE LICENSEES INVESTIGATION OF THE UNIT 3 FAILURE TO FULLY INSERT CONTROL RODS FOLLOWING A SCRAM ON JUNE 28, 1980. OF THE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

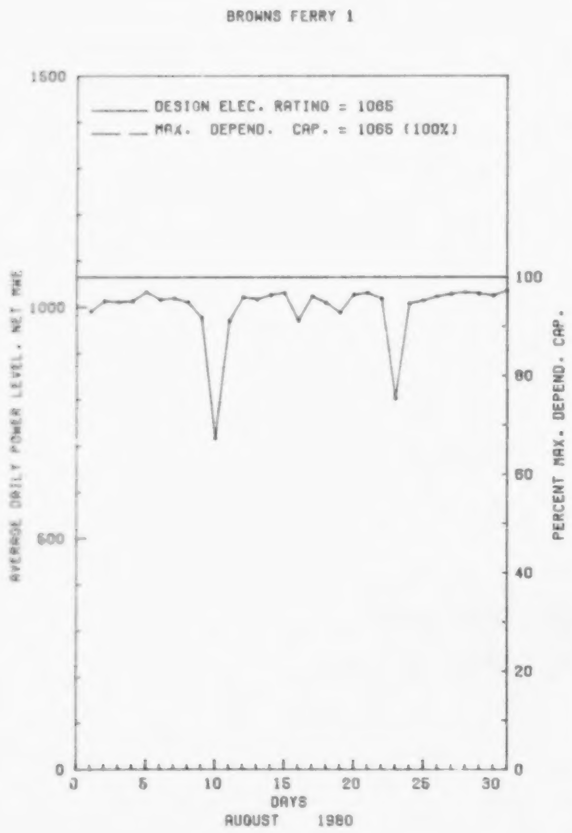
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-050/03L-0	06/24/80	07/18/80	MISWIRED CONTROL VALVE FAST CLOSURE PRESSURE SWITCH
80-051/03L-0	06/26/80	07/18/80	SCRAM DISCHARGE VOLUME HIGH LEVEL SWITCH INOPERABLE
80-052/03L-0	06/28/80	07/25/80	4 KV SHUTDOWN BOARD B TRIPPED
80-055/03L-0	07/08/80	08/06/80	VALVE 79-43 FAILED TO OPEN DURING TEST OF SYSTEM
80-056/03L-0	07/11/80	08/08/80	LINEAR HEAT GENERATION RATE EXCEEDED IN ONE NODE

REVIEWED BY OMPA LOEB: John L Crooks **DATE:** 10/6/80

REVIEWED BY NRR DL: Dick Clark **DATE:** 10-07-80

BROWNS FERRY 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-259
2. Utility Contact: Don Green (205) 729-6846
3. Licensed Thermal Power (MWt): 3293
4. Nameplate Rating (Gross MWe): 1280 x 0.9 = 1152
5. Design Electrical Rating (Net MWe): 1065
6. Maximum Dependable Capacity (Gross MWe): 1098
7. Maximum Dependable Capacity (Net MWe): 1065
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	5,855.0	53,353.0
12. Hours Reactor Was Critical	744.0	3,653.0	31,930.3
13. Reactor Reserve Shutdown Hours	.0	356.4	5,045.9
14. Hours Generator On-Line	744.0	3,524.3	31,137.2
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	2,362,150	10,254,956	84,391,449
17. Gross Electrical Energy (MWH)	766,580	3,297,780	27,816,240
18. Net Electrical Energy (MWH)	741,958	3,194,295	26,996,263
19. Unit Service Factor	100.0	60.2	58.4
20. Unit Availability Factor	100.0	60.2	58.4
21. Unit Capacity Factor (MDC Net)	93.6	51.2	47.5
22. Unit Capacity Factor (DER Net)	93.6	51.2	47.5
23. Unit Forced Outage Rate	.0	11.2	31.1

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
140	08/09/80	S	.0	B	5				DERATED FOR RECIRCULATION PUMP MG SET BRUSH REPLACEMENT.
141	08/18/80	F	.0	B	5				DERATED FOR MAINTENANCE TO CONTROL ROD 18-39.
142	08/22/80	S	.0	H	5				DERATED FOR CONTROL ROD PATTERN ADJUSTMENT.

SUMMARY

BROWNS FERRY 1 OPERATED AT NEAR FULL POWER FOR AUGUST, WITH THREE REDUCTIONS FOR MAINTENANCE.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

BROWNS FERRY 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Alabama/Limestone 10 miles NW of Decatur
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	3293
5.	Design Electrical Rating (MWE Net)	1065
6.	Date of Initial Criticality	July 20, 1974
7.	Date of Commercial Operation	March 1, 1975
8.	Condenser Cooling Method	Once Through With Forced Draft Cooling Towers
9.	Condenser Cooling Water Source	Tennessee River

Utility & Contractor Information

10.	Licensee	Tennessee Valley Authority
11.	Corporate Address	500A Chestnut Street Tower II Chattanooga, Tennessee 37401
12.	Corporate Contact	H. G. Parris, Manager of Power
13.	Architect/Engineer	TVA
14.	Nuclear Steam Supply System	GE
15.	Constructor	TVA

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. Lewis
18.	IE Section Chief	H. C. Dance
19.	IE Resident Inspector	R. Sullivan (205) 729-6196
20.	Licensing Project Manager	R. Clark
21.	Docket Number	50-260
22.	License Number/Date of Issue	DPR-52, 6/28/74
23.	Public Document Room	Athens Public Library South and Forrest Athens, Alabama 35611

INSPECTION STATUS

ENFORCEMENT STATUS

AS REQUIRED BY TECHNICAL SPECIFICATION 6.3.A, DETAILED WRITTEN PROCEDURES SHALL BE PREPARED, APPROVED, AND ADHERED TO FOR SURVEILLANCE AND TESTING REQUIREMENTS. CONTRARY TO THE ABOVE, ON MAY 6, 1980, UNITS 1, 2 AND 3 CONTROL ROOM OPERATORS WERE PERFORMING SURVEILLANCE INSTRUCTION 2, INSTRUMENT CHECKS AND OBSERVATIONS, USING A SECTION WHICH HAD BEEN SUPERCEDED BY AN APPROVED REVISION DATED JULY 18, 1979. (8010 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

ROUTINE OPERATION AT POWER

LAST IE SITE INSPECTION DATE: JUNE 28 - JULY 4, 1980

INSPECTION REPORT NO: 50-260/80-25

SUMMARY

INSPECTION SUMMARY

INSPECTION JUNE 1-30 (80-21): THIS ROUTINE INSPECTION INVOLVED 37 RESIDENT INSPECTOR-HOURS IN THE AREAS OF OPERATIONAL SAFETY MAINTENANCE, NONCOMPLIANCE FOLLOWUP, REPORTABLE OCCURRENCES, REACTOR TRIP FOLLOWUP, PLANT PHYSICAL PROTECTION AND RADIATION SAFETY. OF THE SEVEN AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED IN SIX AREAS. ONE ITEM OF NONCOMPLIANCE WAS FOUND IN ONE AREA (INFRACTION - FAILURE TO TEST A PORTION OF THE REACTOR PROTECTION SYSTEM AFTER MAINTENANCE).

INSPECTION JULY 7-10 (80-22): THE INSPECTION INVOLVED 11 INSPECTOR HOURS ON SITE BY ONE NRC INSPECTOR. THREE INSPECTION HOURS WERE ACCOMPLISHED DURING OFFSHIFT PERIODS. AREAS INSPECTED INCLUDED REVIEW OF SECURITY PLAN; SECURITY ORGANIZATION; PHYSICAL BARRIERS (VITAL AND PROTECTED AREAS); LIGHTING; ASSESSMENT AIDS; ACCESS CONTROLS (IDENTIFICATION, AUTHORIZATION, AND BADGING, SEARCH, ESCORTING AND VITAL AREAS); AND ALARM STATION. THE LICENSEE WAS FOUND TO BE IN COMPLIANCE WITH NRC REQUIREMENTS WITHIN 11 AREAS EXAMINED DURING THE INSPECTION.

INSPECTION JUNE 28 - JULY 4 (80-25): THIS SPECIAL, ANNOUNCED INSPECTION INVOLVED APPROXIMATELY 100 INSPECTOR-HOURS ON SITE IN THE AREAS OF INSPECTION AND EVALUATION OF THE LICENSEES INVESTIGATION OF THE UNIT 3 FAILURE TO FULLY INSERT CONTROL RODS FOLLOWING A SCRAM ON JUNE 28, 1980. OF THE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

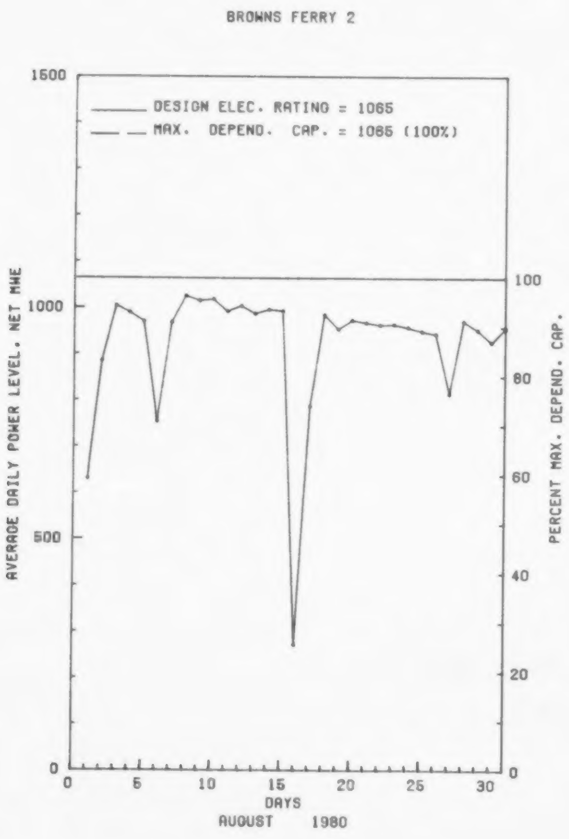
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-024/03L-0	06/14/80	07/22/80	CONTROL ROD 38-27 FAILED TO LATCH
80-026/03L-0	06/27/80	07/28/80	LOST POWER TO 480 VOLT MOTOR OPERATE VALVE BOARD FEEDER BREAKER
80-027/03L-0	07/12/80	08/08/80	MAIN STEAM ISOLATION VALVE LIMIT SWITCH INOPERABLE DURING TEST

REVIEWED BY OMPA LOEB: John L Crooks **DATE:** 10/6/80

REVIEWED BY NRR DL: Dick Clark **DATE:** 10-07-80

BROWNS FERRY 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-260
2. Utility Contact: Don Green (205) 729-6846
3. Licensed Thermal Power (MWt): 3293
4. Nameplate Rating (Gross MWe): 1280 x 0.9 = 1152
5. Design Electrical Rating (Net MWe): 1065
6. Maximum Dependable Capacity (Gross MWe): 1098
7. Maximum Dependable Capacity (Net MWe): 1065
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	5,855.0	48,264.0
12. Hours Reactor Was Critical	733.2	5,185.3	29,669.7
13. Reactor Reserve Shutdown Hours	10.8	649.0	12,423.9
14. Hours Generator On-Line	729.7	5,052.3	28,716.8
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	2,169,170	14,585,799	80,740,799
17. Gross Electrical Energy (MWH)	710,000	4,838,890	26,788,958
18. Net Electrical Energy (MWH)	686,119	4,699,430	26,030,899
19. Unit Service Factor	98.1	86.3	59.5
20. Unit Availability Factor	98.1	86.3	59.5
21. Unit Capacity Factor (MDC Net)	86.6	75.4	50.6
22. Unit Capacity Factor (DER Net)	86.6	75.4	50.6
23. Unit Forced Outage Rate	1.9	12.4	33.3

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUEL OUTAGE SEPTEMBER 5, 1980, 42 DAYS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
152	08/04/80	F	.0	H	5				DERATED DUE TO LOSS OF PLANT PREFERRED POWER.
153	08/05/80	S	0	H	5				DERATED FOR CONTROL ROD PATTERN ADJUSTMENT.
154	08/16/80	F	14.3	G	3				PERSONNEL ERROR WHILE PERFORMING SI 4.2.A-8.

SUMMARY

BROWNS FERRY 2 OPERATED FOR THE ENTIRE MONTH OF AUGUST, WITH TWO REDUCTIONS, AND ONE OUTAGE DUE TO PERSONNEL ERROR.

LEGEND

[1] Type:
F–Forced
S–Scheduled

[2] Reason:
A–Equipment Failure
B–Maintenance or Test
C–Refueling
D–Regulatory Restriction
E–Operator Training & License Examination
F–Administrative
G–Operational Error
H–Other

[3] Method:
1–Manual
2–Manual Scram
3–Automatic Scram
4–Continuations
5–Load Reduction
9–Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

BROWNS FERRY 3

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Alabama/Limestone 10 miles NW of Decatur
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	3293
5.	Design Electrical Rating (MWE Net)	1065
6.	Date of Initial Criticality	August 8, 1976
7.	Date of Commercial Operation	March 1, 1977
8.	Condenser Cooling Method	Once Through With Forced Draft Cooling Tower
9.	Condenser Cooling Water Source	Tennessee River

Utility & Contractor Information

10.	Licensee	Tennessee Valley Authority
11.	Corporate Address	500A Chestnut Street Tower II Chattanooga, Tennessee 37401
12.	Corporate Contact	H. G. Parris, Manager of Power
13.	Architect/Engineer	TVA
14.	Nuclear Steam Supply System	GE
15.	Constructor	TVA

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R.C. Lewis
18.	IE Section Chief	H. C. Dance
19.	IE Resident Inspector	R. Sullivan (205) 729-6196
20.	Licensing Project Manager	R. Clark
21.	Docket Number	50-296
22.	License Number/Date of Issue	DPR-68, 7/2/76
23.	Public Document Room	Athens Public Library South and Forrest Athens, Alabama 35611

INSPECTION STATUS

ENFORCEMENT STATUS

AS REQUIRED BY TECHNICAL SPECIFICATION 6.3.A, DETAILED WRITTEN PROCEDURES SHALL BE PREPARED, APPROVED, AND ADHERED TO FOR SURVEILLANCE AND TESTING REQUIREMENTS. CONTRARY TO THE ABOVE, ON MAY 6, 1980, UNITS 1, 2 AND 3 CONTROL ROOM OPERATORS WERE PERFORMING SURVEILLANCE INSTRUCTION 2, INSTRUMENT CHECKS AND OBSERVATIONS, USING A SECTION WHICH HAD BEEN SUPERCEDED BY AN APPROVED REVISION DATED JULY 10, 1979. (8019 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

FULL POWER OPERATIONS.

LAST IE SITE INSPECTION DATE: JUNE 28 - JULY 4, 1980

INSPECTION REPORT NO: 50-296/80-26

SUMMARY

INSPECTION SUMMARY

INSPECTION JUNE 1-30 (80-22): THIS ROUTINE INSPECTION INVOLVED 37 RESIDENT INSPECTOR-HOURS IN THE AREAS OF OPERATIONAL SAFETY MAINTENANCE, NONCOMPLIANCE FOLLOWUP, REPORTABLE OCCURRENCES, REACTOR TRIP FOLLOWUP, PLANT PHYSICAL PROTECTION AND RADIATION SAFETY. OF THE SEVEN AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED IN SIX AREAS. ONE ITEM OF NONCOMPLIANCE WAS FOUND IN ONE AREA (INFRACTION - FAILURE TO TEST A PORTION OF THE REACTOR PROTECTION SYSTEM AFTER MAINTENANCE).

INSPECTION JULY 7-10 (80-23): THE INSPECTION INVOLVED 10 INSPECTOR HOURS ON SITE BY ONE NRC INSPECTOR. TWO INSPECTION HOURS WERE ACCOMPLISHED DURING OFFSHIFT PERIODS. AREAS INSPECTED INCLUDED REVIEW OF SECURITY PLAN; SECURITY ORGANIZATION; PHYSICAL BARRIERS (VITAL AND PROTECTED AREAS); LIGHTING; ASSESSMENT AIDS; ACCESS CONTROLS (IDENTIFICATION, AUTHORIZATION, AND BADGING, SEARCH, ESCORTING AND VITAL AREAS); AND ALARM STATION. THE LICENSEE WAS FOUND TO BE IN COMPLIANCE WITH NRC REQUIREMENTS WITHIN 11 AREAS EXAMINED DURING THE INSPECTION.

INSPECTION JUNE 28 - JULY 4 (80-26): THIS SPECIAL, ANNOUNCED INSPECTION INVOLVED APPROXIMATELY 100 INSPECTOR-HOURS ON SITE IN THE AREAS OF INSPECTION AND EVALUATION OF THE LICENSEES INVESTIGATION OF THE UNIT 3 FAILURE TO FULLY INSERT CONTROL RODS FOLLOWING A SCRAM ON JUNE 28, 1980. OF THE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

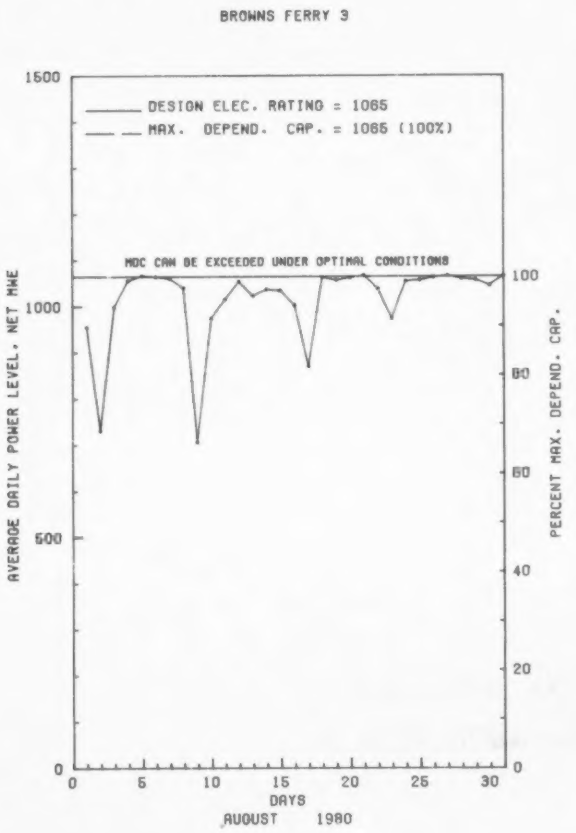
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-024/01T-0	06/28/80	07/18/80	68 CONTROL RODS FAILED TO INSERT DURING A SCRAM
80-026/03L-0	07/14/80	08/11/80	REACTOR CORE ISOLATION COOLING BYPASS VALVE FAILED TO SHUTDOWN DURING TEST
80-027/03L-0	07/14/80	08/18/80	RESIDUAL HEAT REMOVAL PUMP MOTOR TRIPPED ON OVERCURRENT

REVIEWED BY OMPA LOEB: John R Crooks **DATE:** 10/6/80

REVIEWED BY NRR DL: Dick Clark **DATE:** 10-07-80

BROWNS FERRY 3

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-296
2. Utility Contact: Don Green (205) 729-6846
3. Licensed Thermal Power (MWt): 3293
4. Nameplate Rating (Gross MWe): 1280 x 0.9 = 1152
5. Design Electrical Rating (Net MWe): 1065
6. Maximum Dependable Capacity (Gross MWe): 1098
7. Maximum Dependable Capacity (Net MWe): 1065
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	5,855.0	30,719.0
12. Hours Reactor Was Critical	744.0	5,100.2	24,007.9
13. Reactor Reserve Shutdown Hours	.0	101.7	1,760.1
14. Hours Generator On-Line	744.0	5,003.1	23,440.9
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	2,323,303	15,488,186	68,596,0[illegible]2
17. Gross Electrical Energy (MWH)	779,470	5,217,450	22,620,030
18. Net Electrical Energy (MWH)	754,173	5,073,556	21,941,329
19. Unit Service Factor	100.0	85.5	76.[illegible]
20. Unit Availability Factor	100.0	85.5	76.3
21. Unit Capacity Factor (MDC Net)	95.2	81.4	67.1
22. Unit Capacity Factor (DER Net)	95.2	81.4	67.[illegible]
23. Unit Forced Outage Rate	.0	14.5	10.2

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
REFUEL OUTAGE OCTOBER 1980.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
139	08/01/80	S	.0	B	5				DERATED FOR "B" RECIRCULATION PUMP BRUSH REPLACEMENT AND CONTROL ROD PATTERN ADJUSTMENT.
140	08/09/80	S	.0	H	5				DERATED FOR CONTROL ROD SEQUENCE EXCHANGE FROM "B" TO "A".
141	08/16/80	S	.0	H	5				DERATED FOR CONTROL ROD PATTERN ADJUSTMENT.
142	08/22/80	F	.0	B	5				DERATED FOR "B" CONDENSATE BOOSTER PUMP MAINTENANCE.

SUMMARY

BROWNS FERRY 3 OPERATED AT NEAR FULL POWER DURING AUGUST EXCEPT FOR FOUR REDUCTIONS FOR MAINTENANCE AND ADJUSTMENTS.

LEGEND

[1] Type:
F–Forced
S–Scheduled

[2] Reason:
A–Equipment Failure
B–Maintenance or Test
C–Refueling
D–Regulatory Restriction
E–Operator Training & License Examination
F–Administrative
G–Operational Error
H–Other

[3] Method:
1–Manual
2–Manual Scram
3–Automatic Scram
4–Continuations
5–Load Reduction
9–Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

BRUNSWICK 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	North Carolina/Brunswick 3 miles N of Southport
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2436
5.	Design Electrical Rating (MWE Net)	821
6.	Date of Initial Criticality	October 8, 1976
7.	Date of Commercial Operation	March 18, 1977
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Cape Fear River

Utility & Contractor Information

10.	Licensee	Carolina Power and Light Company
11.	Corporate Address	411 Fayetteville Street Raleigh, North Carolina 27602
12.	Corporate Contact	J. A. Jones Executive Vice President
13.	Architect/Engineer	United Engineers & Constructors, Inc.
14.	Nuclear Steam Supply System	GE
15.	Constructor	Brown and Root, Inc.

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R.C. Lewis
18.	IE Section Chief	P.J. Kellogg
19.	IE Resident Inspector	J. E. Outzs (919) 457-9531
20.	Licensing Project Manager	J. Hannon
21.	Docket Number	50-325
22.	License Number/Date of Issue	DPR-71, 9/8/76
23.	Public Document Room	Southport-Brunswick County Library 109 W. Moore Street Southport, North Carolina 28461

INSPECTION STATUS

ENFORCEMENT STATUS

FAILURE TO FOLLOW DRAWINGS IN THAT 17 FEED-THRU CONDUCTORS OF 480V ELECTRICAL PENETRATION WERE NOT TERMINATED IN ACCORDANCE WITH REQUIREMENTS.
(8021 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

ROUTINE POWER OPERATION AFTER COMPLETION OF REFUELING OPERATIONS.

LAST IE SITE INSPECTION DATE: AUGUST 1-31, 1980

INSPECTION REPORT NO: 50-325/80-35

SUMMARY

INSPECTION SUMMARY

INSPECTION JUNE 21-24 (80-27): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 16 INSPECTOR-HOURS ON SITE IN THE AREAS OF PERIODIC TESTING OF DIESEL GENERATOR UNITS USED AS ONSITE ELECTRIC POWER SYSTEMS, LICENSEE ACTIONS ON PREVIOUSLY IDENTIFIED ITEMS, AND REVIEW OF DIESEL GENERATOR TESTING REQUIREMENTS AT IE HEADQUARTERS' REQUEST. OF THE THREE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN TWO AREAS; ONE ITEM OF NONCOMPLIANCE WAS FOUND IN ONE AREA (DEFICIENCY - FAILURE TO PROVIDE COMPLETE TEST REQUIREMENTS AND INSTRUCTIONS).

REPORTS RECEIVED FROM LICENSEE

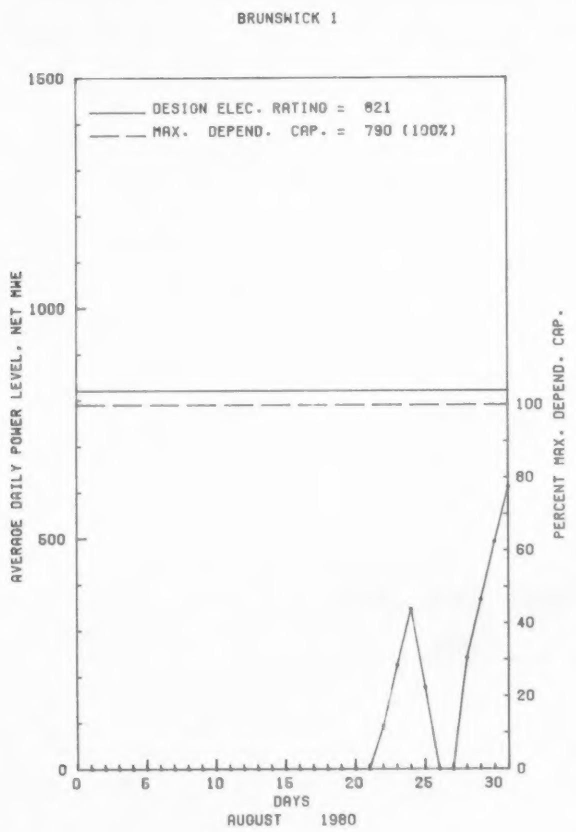
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-034/ 03L-0	08/04/80	08/19/80	OVEREXTENDED BERGEN PATTERSON SNUBBER
80-053/ 03L-0	05/07/80	07/11/80	BOTH INNER AND OUTER DOORS OF 50 FT ELEVATION AIR LOCK OPEN
80-056 03L-0	07/01/80	07/23/80	APRM BAND INOPERATIVE DUE TO PULLED LOOSE CONNECTORS
80-061/ 03L-0	08/01/80	08/04/80	INADEQUATE SUPPORTS ON CRD SYSTEM
80-062/ 03L-0	07/20/80	08/14/80	CONTROL ROD POSITION INDICATION PROBLEMS ON FOUR RODS
80-064/ 03L-0	07/23/80	08/01/80	LOW VOLTAGE ON 1B1 PLANT STATION BATTERY

REVIEWED BY OMPA LOEB: John R Crooks DATE: 10/6/80

REVIEWED BY NRR DL: John N. Hannon DATE: 10/7/80

BRUNSWICK 1

AVERAGE POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-325
2. Utility Contact: E. Willis (919) 457-9521
3. Licensed Thermal Power (MWt): 2436
4. Nameplate Rating (Gross MWe): 963 x 0.9 = 867
5. Design Electrical Rating (Net MWe): 821
6. Maximum Dependable Capacity (Gross MWe): 815
7. Maximum Dependable Capacity (Net MWe): 790
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	5,855.0	30,312.0
12. Hours Reactor Was Critical	256.8	3,596.9	20,986.5
13. Reactor Reserve Shutdown Hours	.0	.0	1,647.1
14. Hours Generator On-Line	166.9	3,315.2	19,648.0
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	190,222	6,428,156	40,257,652
17. Gross Electrical Energy (MWH)	[illegible]4,376	2,151,850	13,390,569
18. Net Electrical Energy (MWH)	55,025	2,066,435	12,874,245
19. Unit Service Factor	22.4	56.6	64.8
20. Unit Availability Factor	22.4	56.6	64.8
21. Unit Capacity Factor (MDC Net)	9.4	44.7	53.8
22. Unit Capacity Factor (DER Net)	9.0	43.0	51.7
23. Unit Forced Outage Rate	26.2	11.5	20.0

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
019	05/26/80	S	518.0	C	3		RC	FUELXX	A PLANNED REFUELING/MAINTENANCE OUTAGE WAS EXTENDED BEYOND ITS SCHEDULED DATE BECAUSE OF UNANTICIPATED MAINTENANCE AND REGULATORY PROBLEMS.
020	08/25/80	F	59.1	H	2	1-80-65	MA	ZZZZZZ	REACTOR COOLANT CONDUCTIVITY EXCEEDED 2 UMHO/CM WHICH IS THE LIMIT PERMITTED BY TECHNICAL SPECIFICATION. OPERATION BETWEEN 2 UMHO/CM AND 10 UMHO/CM CAN CONTINUE UP TO 24 HOURS. AFTER THE 24 HOUR ALLOWABLE TIME, THE UNIT MUST BE SHUTDOWN IN THE NEXT 12 HOURS.

SUMMARY

BRUNSWICK UNIT 1 CAME ON-LINE ON THE 22ND FROM A REFUELING OUTAGE. A FORCED OUTAGE OCCURRED ON AUGUST 25 DUE TO REACTOR COOLANT CONDUCTIVITY LIMIT BEING EXCEEDED. UNIT RETURNED ON-LINE ON THE 28TH.

LEGEND

[1] Type:
F–Forced
S–Scheduled

[2] Reason:
A–Equipment Failure
B–Maintenance or Test
C–Refueling
D–Regulatory Restriction
E–Operator Training & License Examination
F–Administrative
G–Operational Error
H–Other

[3] Method:
1–Manual
2–Manual Scram
3–Automatic Scram
4–Continuations
5–Load Reduction
9–Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

best copy available

BRUNSWICK 2

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	North Carolina/Brunswick 3 miles N of Southport
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2436
5.	Design Electrical Rating (MWE Net)	821
6.	Date of Initial Criticality	March 20, 1975
7.	Date of Commercial Operation	November 3, 1975
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Cape Fear River

Utility & Contractor Information

10.	Licensee	Carolina Power and Light Company
11.	Corporate Address	411 Fayetteville Street Raleigh, North Carolina 27602
12.	Corporate Contact	J. A. Jones Executive Vice President
13.	Architect/Engineer	United Engineers & Constructors, Inc.
14.	Nuclear Steam Supply System	GE
15.	Constructor	Brown and Root, Inc.

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R.C. Lewis
18.	IE Section Chief	P.J. Kellogg
19.	IE Resident Inspector	J. E. Outzs (919) 457-9531
20.	Licensing Project Manager	J. Hannon
21.	Docket Number	50-324
22.	License Number/Date of Issue	DPR-62, 12/27/74
23	Public Document Room	Southport-Brunswick County Library 109 W. Moore Street Southport, North Carolina 28461

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO THE TECHNICAL SPECIFICATION 6.8.1.C PERIODIC TEST 12.1.2 AND 12.1.1A APPROVED ON APRIL 9 AND MAY 21, 1980. DO NOT COVER THE COMPLETE LOADING ACTIVITIES REQUIRED BY TECHNICAL SPECIFICATIONS AND FSAR IN THAT THE PROCEDURE DOES NOT INCLUDE STARTUP OF ONE FIRE PUMP MOTOR AND A TOTAL OF TWENTY MOTOR-OPERATED VALVE MOTORS. HOWEVER, THE LICENSEE TEMPORARILY REVISED THE PROCEDURE TO INCLUDE THE FIRE PUMP LOADING AT THE TIME OF THE JUNE 21 TEST AS A PARTIAL RESOLUTION OF NRC CONCERNS. THE LICENSEE MADE FURTHER COMMITMENTS TO PERMANENTLY REVISE THE PERIODIC TEST PROCEDURES, TO RESOLVE THE TOTAL CONCERN, PRIOR TO THE PERFORMANCE OF THE NEXT SCHEDULED SURVEILLANCE TEST ON EITHER UNIT. (8024 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

UNIT SHUTDOWN FOR REFUELING AND TORUS MODIFICATION.

LAST IE SITE INSPECTION DATE: AUGUST 1-31, 1980

INSPECTION REPORT NO: 50-324/80-30

SUMMARY

INSPECTION SUMMARY

INSPECTION JUNE 21-24 (80-27): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 16 INSPECTOR-HOURS ON SITE IN THE AREAS OF PERIODIC TESTING OF DIESEL GENERATOR UNITS USED AS ONSITE ELECTRIC POWER SYSTEMS, LICENSEE ACTIONS ON PREVIOUSLY IDENTIFIED ITEMS, AND REVIEW OF DIESEL GENERATOR TESTING REQUIREMENTS AT IE HEADQUARTERS' REQUEST. OF THE THREE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN TWO AREAS; ONE ITEM OF NONCOMPLIANCE WAS FOUND IN ONE AREA (DEFICIENCY - FAILURE TO PROVIDE COMPLETE TEST REQUIREMENTS AND INSTRUCTIONS).

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

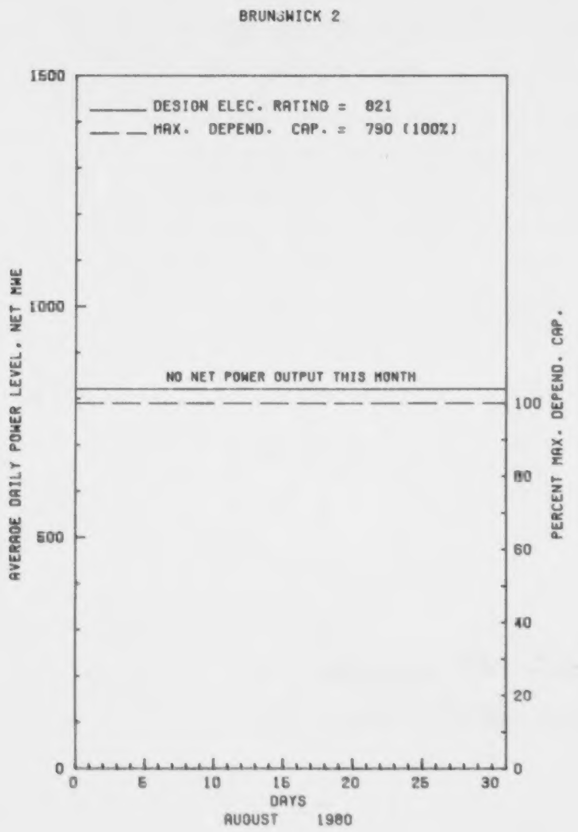
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-039/ 03L-0	07/21/80	08/06/80	INOPERABLE SNUBBER
80-042/ 03L-0	07/20/80	08/01/80	REACTOR LO PRESS SWITCH 1 OUT OF CALIBRATION
80-045/ 03L-0	07/07/80	08/04/80	A AND D SRM'S INOPERABLE
80-048/ 03L-0	06/23/80	07/16/80	REACTOR VESSEL LOW SWITCH ACTUATED DURING PT AND WOULD NOT RESET
80-049 03L-0	07/01/80	08/12/80	HIGH DRYWELL PRESSURE SWITCH OUT OF CALIBRATION
80-055/ 01T-0	07/28/80	08/11/80	FAILED ANCHOR BOLTS ON CRD PIPING

REVIEWED BY OMPA LOEB: John L Crooks DATE: 10/6/80

REVIEWED BY NRR DL: ____________ DATE: ____________

BRUNSWICK 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-324
2. Utility Contact: E. Willis (919) 457-9521
3. Licensed Thermal Power (MWt): 2436
4. Nameplate Rating (Gross MWe): 963 x 0.9 = 867
5. Design Electrical Rating (Net MWe): 821
6. Maximum Dependable Capacity (Gross MWe): 815
7. Maximum Dependable Capacity (Net MWe): 790
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	5,855.0	42,336.0
12. Hours Reactor Was Critical	.0	1,365.7	26,689.6
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	.0	1,228.1	25,102.1
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	2,604,960	46,339,422
17. Gross Electrical Energy (MWH)	0	872,613	15,487,687
18. Net Electrical Energy (MWH)	0	842,126	14,882,598
19. Unit Service Factor	.0	21.0	59.3
20. Unit Availability Factor	.0	21.0	59.3
21. Unit Capacity Factor (MDC Net)	.0	18.2	44.5
22. Unit Capacity Factor (DER Net)	.0	17.5	42.8
23. Unit Forced Outage Rate	.0	8.1	12.8

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 09/04/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	03/01/80	S	744.0	C	4		RC	FUELXX	REFUELING OUTAGE CONTINUES.

SUMMARY

THE UNIT REMAINED SHUTDOWN THROUGHOUT THE REPORT PERIOD FOR REFUELING.

LEGEND

[1] Type:
F–Forced
S–Scheduled

[2] Reason:
A–Equipment Failure
B–Maintenance or Test
C–Refueling
D–Regulatory Restriction
E–Operator Training & License Examination
F–Administrative
G–Operational Error
H–Other

[3] Method:
1–Manual
2–Manual Scram
3–Automatic Scram
4–Continuations
5–Load Reduction
9–Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

CALVERT CLIFFS 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Maryland/Calvert 40 miles S of Annapolis
2.	Electric Reliability Council	Mid-Atlantic Area Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2700
5.	Design Electrical Rating (MWE Net)	845
6.	Date of Initial Criticality	October 7, 1974
7.	Date of Commercial Operation	May 8, 1975
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Chesapeake Bay

Utility & Contractor Information

10.	Licensee	Baltimore Gas and Electric Company
11.	Corporate Address	P. O. Box 1475 Baltimore, Maryland 21203
12.	Corporate Contact	A. E. Lundvall, Jr. Vice President-Supply
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Combustion Engineering
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Section Chief	E. C. McCabe, Jr.
19.	IE Resident Inspector	R. Architzel (301) 586-2626
20.	Licensing Project Manager	E. L. Conner, Jr.
21.	Docket Number	50-317
22.	License Number/Date of Issue	DPR-53, 7/31/74
23.	Public Document Room	Calvert County Library Prince Frederick, Maryland 20678

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO 10 CFR 50.72(A)(6) THE LICENSEE DID NOT NOTIFY THE NRC WITHIN THE REQUIRED TIME FRAME (1 HOUR) FOLLOWING DISCOVERY THAT THE AUXILIARY FEEDWATER SYSTEM WAS INOPERABLE. (DISCOVERED 1:00 A.M., MAY 21, 1980 REPORTED 1255 P.M., MAY 21, 1980).
(8006 INFRACTION)

CONTRARY TO T.S. 3.7.1.2, THE AUXILIARY FEEDWATER SYSTEM WAS INOPERABLE WITH THE REACTOR IN MODE 3 BETWEEN ABOUT 1000 P.M. ON MAY 20 AND ABOUT 1:00 A.M. ON MAY 21, 1980.
(8006 INFRACTION)

CONTRARY TO LICENSE CONDITION 2.C.(4), FIRE PROTECTION ADMINISTRATIVE CONTROLS WERE NOT IMPLEMENTED, IN THAT, A BOTTLE OF ACETYLENE GAS WAS NOT STORED BUT LEFT FREE STANDING OUTSIDE THE INTAKE STRUCTURE.
(8006 DEFICIENCY)

CONTRARY TO LICENSE CONDITION 2.C.(4) THE LICENSEE FAILED TO IMPLEMENT FIRE PROTECTION ADMINISTRATIVE CONTROLS IN THAT EVIDENCE OF SMOKING WAS OBSERVED IN THE CONTROL ROOM PANEL BACKS ON MAY 14 AND 28, 1980.
(8006 INFRACTION)

CONTRARY TO T.S. 6.8.1, THE CONTROL ROOM OPERATORS LOG AND SHIFT SUPERVISORS LOG WERE NOT FILLED IN AS REQUIRED FOLLOWING DISCOVERY OF THE AUXILIARY FEEDWATER SYSTEM INOPERABILITY ON MAY 21, 1980.
(8006 DEFICIENCY)

CONTRARY TO PART H3 OF THE APPROVED PHYSICAL SECURITY PLAN, THE SECURITY FORCE FAILED TO RESPOND TO A VITAL AREA ACCESS ALARM ON MAY 19, 1980.
(8006 INFRACTION)

CONTRARY TO THE APPROVED PHYSICAL SECURITY PLAN, PART H.3, THE SECURITY FORCE FAILED TO RESPOND TO A VITAL AREA ACCESS ALARM ON MAY 15, 1980.
(8006 INFRACTION)

CONTRARY TO PART H3 OF THE APPROVED PHYSICAL SECURITY PLAN, THE SECURITY FORCE FAILED TO RESPOND TO A VITAL AREA ACCESS ALARM ON MAY 16, 1980.
(8006 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

- FACILITY ITEMS (PLANS AND PROCEDURES):

- SPENT FUEL POOL MODIFICATION PLANNED FOR IMPLEMENTATION IN SEPTEMBER TO INCREASE STORAGE CAPACITY TO 830 FUEL ASSEMBLIES.

PLANT STATUS:

No significant new information.

- LAST IE SITE INSPECTION DATE: 8/1-31/80

- INSPECTION REPORT NO: 50-317/80-14

SUMMARY

INSPECTION SUMMARY

- 50-317/80-06 - MAY 1-31: ROUTINE, ONSITE REGULAR AND BACKSHIFT INSPECTION BY THE RESIDENT INSPECTOR (35 HRS). AREAS INSPECTED INCLUDED THE CONTROL ROOM AND THE ACCESSIBLE PORTIONS OF THE AUXILIARY, TURBINE, SERVICE, AND INTAKE BUILDINGS; RADIATION PROTECTION; PHYSICAL SECURITY; FIRE PROTECTION; PLANT OPERATING RECORDS; ADMINISTRATIVE CONTROLS RELATING TO DEFEAT OF SAFETY ACTUATION SIGNALS, AND A MEETING WITH A REPRESENTATIVE OF THE FEDERAL BUREAU OF INVESTIGATION. EIGHT ITEMS OF NONCOMPLIANCE WERE IDENTIFIED: (1) INOPERABILITY OF THE AFWS; (2) FAILURE TO REPORT AFWS INOPERABILITY; (3) FAILURE TO LOG AFWS INOPERABILITY; (4) FAILURE TO PREVENT SMOKING BEHIND CONTROL ROOM CONTROL PANELS; (5) FAILURE TO IMPLEMENT FIRE PROTECTION ADMINISTRATION - ACETYLENE GAS BOTTLE NOT SECURED; (6-7-8) THREE FAILURES TO RESPOND TO AREA ACCESS ALARMS ON MAY 15, 1980, MAY 16, 1980, AND MAY 19, 1980.

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-032/03L	07/01/80	08/01/80	NO. 12 SERVICE WATER HEAT EXCHANGER OUTLET VALVE WOULD NOT STROKE FULLY OPEN. CAUSED BY CORROSION PRODUCT BUILDUP ON ACTUATOR PISTON ROD.
80-033/03L	07/28/80	07/29/80	NO. 14 CONTAINMENT AIR COOLER WOULD NOT SHIFT TO SLOW SPEED (LOCA SPEED) FROM THE CONTROL ROOM. CAUSED BY A RELAY (ROWAN CONTROLLER) FAILING TO DROP OUT WHEN DEENERGIZED.
80-034/01T	07/16/80	07/30/80	CONTROL ROOM AIR CONDITIONERS TRIPPED OFF LINE DURING HIGH SUMMER HEATING LOAD. CORRECTIVE ACTION TAKEN TO INCREASE LOAD CARRYING CAPACITY BEFORE TRIPPING.
80-035/01T	07/27/80	08/08/80	LICENSED THERMAL POWER EXCEEDED BY 24 MW (2700 LIMIT) DUE TO TUBE LEAKAGE IN SG BLOWDOWN HEAT EXCHANGER RESULTING IN IMPROPER FLOW INPUT TO CALORIMETRIC.
80-036/03L	07/21/80	08/20/80	NO. 11 DIESEL GENERATOR LOST SPEED AND VOLTAGE CONTROL CAUSED BY WATER SPRAY ON CONTROL CABINET DURING HEAT EXCHANGER VENTING.
80-037/01T	07/30/80	08/13/80	DIESEL AIR START PRESSURE SWITCHES INSTRUMENT TUBING NOT SEISMICALLY QUALIFIED DISCOVERED DURING REVIEW OF AS-BUILT CONFIGURATION (IEB 79-14)
80-041/01T	08/12/80	08/26/80	BOTH SERVICE WATER SUBSYSTEMS OPERATED IN A DEGRADED CONDITION WHEN AIR INGRESS OCCURRED FROM NO. 11 INSTRUMENT AIR COMPRESSOR. SYSTEM PRESSURE REMAINED NORMAL.

50-317/80-11 - JUL 1-31: ROUTINE, ONSITE REGULAR AND BACKSHIFT INSPECTION BY THE RESIDENT INSPECTOR (29 HRS). AREAS INSPECTED INCLUDED THE CONTROL ROOM AND THE ACCESSIBLE PORTIONS OF THE AUXILIARY, TURBINE, SERVICE, AND INTAKE BUILDINGS; RADIATION PROTECTION; PHYSICAL SECURITY; FIRE PROTECTION; PLANT OPERATING RECORDS; MAINTENANCE ACTIVITIES; IE CIRCULAR REVIEW; IE BULLETIN REVIEW; REVIEW OF LICENSEE REPORTS AND EVENTS. NO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED.

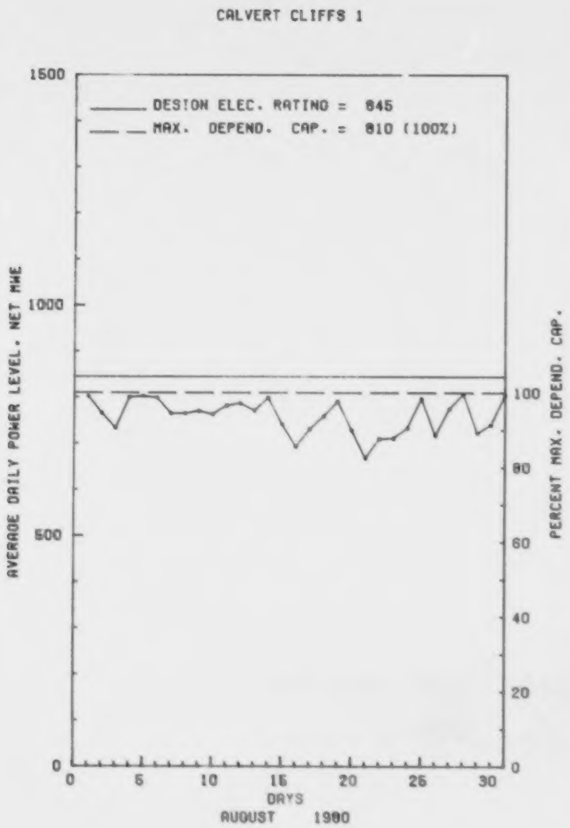
50-317/80-12 - JUL 15-18: ROUTINE, UNANNOUNCED PHYSICAL PROTECTION INSPECTION BY ONE REGION-BASED INSPECTOR (45 HRS) INCLUDING REVIEW OF: SECURITY PLAN; SECURITY ORGANIZATION; PROTECTED AREA PHYSICAL BARRIERS; VITAL AREA PHYSICAL BARRIERS; ACCESS CONTROL (VITAL AREA); ALARM STATIONS; DETECTION AIDS; ASSESSMENT AIDS; COMMUNICATIONS; RESPONSE; FOLLOWUP ON IE BULLETINS #77-08 (LOCKING MECHANISMS) AND #79-16 (VITAL AREA ACCESS CONTROLS AND A FOLLOWUP ON PREVIOUS ITEMS. FOUR ITEMS OF NONCOMPLIANCE WERE IDENTIFIED: (1) FAILURE TO ISSUE WRITTEN PROCEDURE; (2) FAILURE TO MAINTAIN CONTINUOUS COMMUNICATIONS; (3) FAILURE TO CONTROL ACCESS TO THE PROTECTED AREA; (4) FAILURE TO SECURE VITAL AREA PORTAL.

REVIEWED BY OMPA LOEB: MR Beebe **DATE:** 10/06/80

REVIEWED BY NRR DL: **DATE:** 10-7-80

CALVERT CLIFFS 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-317
2. Utility Contact: S. D. Merson (301) 234-5240
3. Licensed Thermal Power (MWt): 2700
4. Nameplate Rating (Gross MWe): 1020 x 0.9 = 918
5. Design Electrical Rating (Net MWe): 845
6. Maximum Dependable Capacity (Gross MWe): 845
7. Maximum Dependable Capacity (Net MWe): 810
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	5,855.0	46,620.0
12. Hours Reactor Was Critical	744.0	5,291.0	37,730.6
13. Reactor Reserve Shutdown Hours	.0	38.3	1,098.8
14. Hours Generator On-Line	744.0	5,221.6	36,923.6
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,900,363	12,448,113	87,785,101
17. Gross Electrical Energy (MWH)	592,165	3,916,949	28,729,479
18. Net Electrical Energy (MWH)	564,729	3,721,688	27,370,894
19. Unit Service Factor	100.0	89.2	79.2
20. Unit Availability Factor	100.0	89.2	79.2
21. Unit Capacity Factor (MDC Net)	93.7	78.5	72.5
22. Unit Capacity Factor (DER Net)	89.8	75.2	69.5
23. Unit Forced Outage Rate	.0	3.5	8.3

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
REFUELING, INSPECTION & MAINTENANCE - 10/17/80.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	NONE								

SUMMARY

CALVERT CLIFFS 1 OPERATED FOR THE ENTIRE MONTH WITH NO OUTAGES OR REDUCTIONS.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source

CALVERT CLIFFS 2

NRC OWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Maryland/Calvert 40 miles S of Annapolis
2.	Electric Reliability Council	Mid-Atlantic Area Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2700
5.	Design Electrical Rating (MWE Net)	845
6.	Date of Initial Criticality	November 30, 1976
7.	Date of Commercial Operation	April 1, 1977
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Chesapeake Bay

Utility & Contractor Information

10.	Licensee	Baltimore Gas and Electric Company
11.	Corporate Address	P. O. Box 1475 Baltimore, Maryland 21203
12.	Corporate Contact	A. E. Lundvall, Jr. Vice President-Supply
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Combustion Engineering
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	I E Section Chief	E. C. MCCABE, JR.
19.	IE Resident Inspector	R. Architzel (301) 586-2626
20.	Licensing Project Manager	E. L. Conner, Jr.
21.	Docket Number	50-318
22.	License Number/Date of Issue	DPR-69, 8/13/76
23.	Public Document Room	Calvert County Library Prince Frederick, Maryland 20678

INSPECTION STATUS

ENFORCEMENT STATUS

LICENSE CONDITION 2.C AND LICENSEE REQUIREMENTS FOR STORAGE OF FLAMMABLE GAS CONTAINERS. AN ACETYLENE GAS BOTTLE WAS NOT PROPERLY SECURED. IN THAT IT WAS STANDING FREE OUTSIDE THE INTAKE STRUCTURE ENTRANCE.
(8006 DEFICIENCY)

CONTRARY TO LICENSE CONDITION 2.C(6) THE LICENSEE FAILED TO IMPLEMENT FIRE PROTECTION ADMINISTRATIVE CONTROLS IN THAT EVIDENCE OF SMOKING WAS OBSERVED IN THE CONTROL ROOM PANEL BACKS ON MAY 14 AND 28, 1980.
(8006 INFRACTION)

CONTRARY TO THE APPROVED PHYSICAL SECURITY PLAN, THE SECURITY FORCE DID NOT RESPOND TO A VITAL AREA ACCESS ALARM ON MAY 15, 1980.
(8006 INFRACTION)

CONTRARY TO THE APPROVED PHYSICAL SECURITY PLAN, SECTION H.3, THE SECURITY FORCE FAILED TO RESPOND TO A VITAL AREA ACCESS ALARM ON MAY 16, 1980.
(3006 INFRACTION)

CONTRARY TO THE APPROVED PHYSICAL SECURITY PLAN, SECTION H.3, THE SECURITY FORCE DID NOT RESPOND TO A VITAL AREA ACCESS ALARM ON MAY 16, 1980.
(3006 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS:

No significant new information.

LAST IE SITE INSPECTION DATE: 8/1-31/80

INSPECTION REPORT NO: 50-318/80-13

SUMMARY

INSPECTION SUMMARY

50-318/80-06 - MAY 1-31: ROUTINE, ONSITE REGULAR AND BACKSHIFT INSPECTION BY THE RESIDENT INSPECTOR (30 HRS). AREAS INSPECTED INCLUDED THE CONTROL ROOM AND THE ACCESSIBLE PORTIONS OF THE AUXILIARY, TURBINE, SERVICE, AND INTAKE BUILDINGS; RADIATION PROTECTION; PHYSICAL SECURITY; FIRE PROTECTION; PLANT OPERATING RECORDS; ADMINISTRATIVE CONTROLS RELATING TO DEFEAT OF SAFETY ACTUATION SIGNALS, AND A MEETING WITH A REPRESENTATIVE OF THE FEDERAL BUREAU OF INVESTIGATION. FOUR ITEMS OF NONCOMPLIANCE WERE IDENTIFIED: ONE FAILURE TO IMPLEMENT FIRE PROTECTION ADMINISTRATIVE CONTROLS (SMOKING BEHIND CONTROL ROOM CONTROL PANELS); AND, THREE FAILURES TO RESPOND TO SECURITY ALARMS: MAY 15, 1980, MAY 16, 1980 AND MAY 19, 1980.

50-318/80-10 - JUL 1-31: ROUTINE, ONSITE REGULAR AND BACKSHIFT INSPECTION BY THE RESIDENT INSPECTORS (29 HRS). AREAS INSPECTED INCLUDED THE CONTROL ROOM AND THE ACCESSIBLE PORTIONS OF THE AUXILIARY, TURBINE, SERVICE, AND INTAKE BUILDINGS; RADIATION PROTECTION; PHYSICAL SECURITY; FIRE PROTECTION; PLANT OPERATING RECORDS; MAINTENANCE ACTIVITIES; IE CIRCULAR REVIEW; IE BULLETIN REVIEW; REVIEW OF LICENSEE REPORTS AND EVENTS. NO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED.

50-318/80-11 - JUL 13-18: ROUTINE, UNANNOUNCED PHYSICAL PROTECTION INSPECTION BY ONE REGION-BASED INSPECTOR (48 HRS), INCLUDING: A REVIEW OF: SECURITY PLAN; SECURITY ORGANIZATION; PROTECTED AREA PHYSICAL BARRIERS; VITAL AREA PHYSICAL BARRIERS; ACCESS CONTROL (VITAL AREAS); ALARM STATIONS; DETECTION AIDS; ASSESSMENT AIDS; COMMUNICATIONS; RESPONSE; FOLLOWUP ON IE BULLETINS #77-08 (LOCKING MECHANISMS) AND #79-16 (VITAL AREA ACCESS CONTROLS) AND FOLLOWUP ON OTHER PREVIOUS ITEMS. FOUR ITEMS OF NONCOMPLIANCE WERE IDENTIFIED: FAILURE TO ISSUE WRITTEN PROCEDURE; FAILURE TO MAINTAIN CONTINUOUS COMMUNICATIONS; FAILURE TO CONTROL ACCESS TO THE PROTECTED AREA; AND, FAILURE TO SECURE VITAL AREA PORTAL.

REPORTS RECEIVED FROM LICENSEE

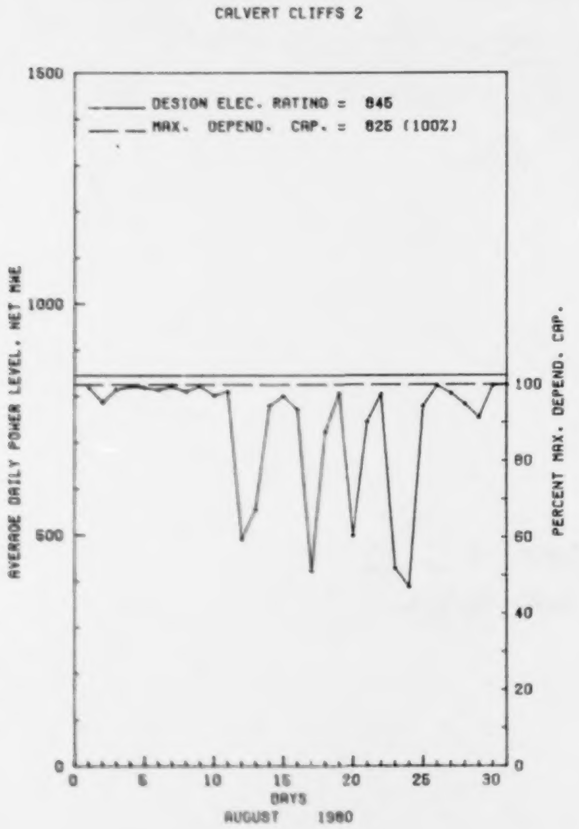
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-032/ 01T	07/23/80	08/06/80	EMERGENCY POWER SUPPLY FOR NO. 21 CHG PUMP REMOVED FROM SERVICE WHILE TWO REDUNDANT PUMPS WERE OOS FOR MAINTENANCE.
80-033/ 03L	07/16/80	08/13/80	SPRAY VALVE MINIMUM FLOW BYPASS LEAKOFF ISOLATION VALVE PACKING LEAKAGE.
80-034/ 01T	07/27/80	08/08/80	LICENSED THERMAL POWER (2700 MW) EXCEEDED BY 8 MW DUE TO TUBE LEAKAGE IN SG BLOWDOWN HEAT EXCHANGER RESULTING IN IMPROPER FLOW INPUT TO COMPUTER CALORIMETRIC.
80-035/ 01T	07/30/80	08/13/80	DIESEL AIR START PRESSURE SWITCHES INSTRUMENT TUBING NOT SEISMICALLY QUALIFIED. DISCOVERED DURING REVIEW OF AS-BUILT CONFIGURATION (IEB 79-14).

REVIEWED BY OMPA LOEB: MR Beebe DATE: 10/06/80

REVIEWED BY NRR DL: DATE: 10-7-80

CALVERT CLIFFS 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-318
2. Utility Contact: S. D. Merson (301) 234-5240
3. Licensed Thermal Power (MWt): 2700
4. Nameplate Rating (Gross MWe): 1012 x 0.9 = 911
5. Design Electrical Rating (Net MWe): 845
6. Maximum Dependable Capacity (Gross MWe): 860
7. Maximum Dependable Capacity (Net MWe): 825
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

ITEM 21 CALCULATED WITH A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	5,855.0	29,975.0
12. Hours Reactor Was Critical	728.5	5,600.3	25,545.2
13. Reactor Reserve Shutdown Hours	13.0	22.5	435.6
14. Hours Generator On-Line	723.9	5,569.9	25,178.4
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,809,298	14,189,344	62,433,576
17. Gross Electrical Energy (MWH)	572,695	4,680,456	20,685,678
18. Net Electrical Energy (MWH)	545,860	4,473,044	19,730,064
19. Unit Service Factor	97.3	95.1	84.0
20. Unit Availability Factor	97.3	95.1	84.0
*21. Unit Capacity Factor (MDC Net)	88.9	92.6	80.6
22. Unit Capacity Factor (DER Net)	86.8	90.4	77.9
23. Unit Forced Outage Rate	2.7	1.0	5.7

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-04	08/12/80	F	14.8	A	1		XX	PUMPXX	LOSS OF NO. 22 CIRCULATING WATER PUMP.
80-05	08/20/80	F	5.3	G	3		XX	ZZZZZZ	REACTOR TRIPPED ON HIGH PRESSURIZER PRE. URE WHEN A TECHNICIAN INADVERTENTLY INITIATED STEAM GENERATOR ISOLATION SIGNAL.

SUMMARY

CALVERT CLIFFS 2 OPERATED FOR THE MONTH OF AUGUST WITH TWO OUTAGES DUE TO EQUIPMENT FAILURE AND OPERATIONAL ERROR.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

COOK 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Michigan/Berrian 11 miles S of Benton Harbor
2.	Electric Reliability Council	East Central Area Reliability Coordination Agreement
3.	Type of Reactor	PWR
4.	Power Level (MWT)	3250
5.	Design Electrical Rating (MWE Net)	1054
6.	Date of Initial Criticality	January 18, 1975
7.	Date of Commercial Operation	August 27, 1975
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10.	Licensee	AEP Corporation
11.	Corporate Address	Indiana & Michigan Electric Co. 2 Broadway New York, New York 10004
12.	Corporate Contact	John Dolan Senior Executive V. P. Eng.
13.	Architect/Engineer	AEP
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Indiana & Michigan Power Co.

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	D.C. Boyd
19.	IE Resident Inspector	E. Swanson (616) 465-6775
20.	Licensing Project Manager	S. Miner
21.	Docket Number	50-315
22.	License Number/Date of Issue	DPR-58 issued 10/25/74
23.	Public Document Room	Maude Preston Palenske Memorial Library 500 Market Street St. Joseph, Michigan 49085

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENTS

New thermal sleeves and elbows, designed by Westinghouse Electric and installed by Power Systems, Inc. were replaced on all Unit 1 steam generators. Low pressure turbines A and B were ultrasonically examined by General Electric Company, for keyway and erosion cracking. No indications reported. (80-10)

PLANT STATUS

Operating routinely.

LAST IE SITE INSPECTION DATE: July 22-25, 1980

INSPECTION REPORT NO: 80-12

SUMMARY

INSPECTION SUMMARY

Inspection on June 11-13, 17-20, 25-26; July 8-11 and 24, (80-10): Review of ISI program, procedures, work activities, and documentation for this refueling outage; follow-up on IE Bulletin 79-37, Cracking in L.P. Turbine Discs; follow-up on potential welding deficiencies in tanks fabricated by Graver Tank Manufacturing Company; review of procedures, personnel qualifications, and installation activities for feedwater thermal sleeve elbows on all Unit 1 steam generators. The inspection involved 92 inspector hours onsite by one NRC inspector. Of the areas examined, one apparent item of noncompliance was identified in one area (failure to develop and implement an indoctrination and training program for QC inspectors).

Inspection on July 22-25, (80-12): Routine, unannounced inspection of operational radiation protection program during refueling and maintenance, including: organization and staffing; training; radiation protection procedures; planning, preparation, and ALARA; exposure control; posting and control; material control; surveys; followup on IE Bulletin and Circulars; and previous noncompliance and unresolved item review. The inspection involved 36 inspector hours onsite by one NRC inspector. No items of noncompliance were identified in any of the 10 areas inspected.

REPORTS RECEIVED FROM LICENSEE

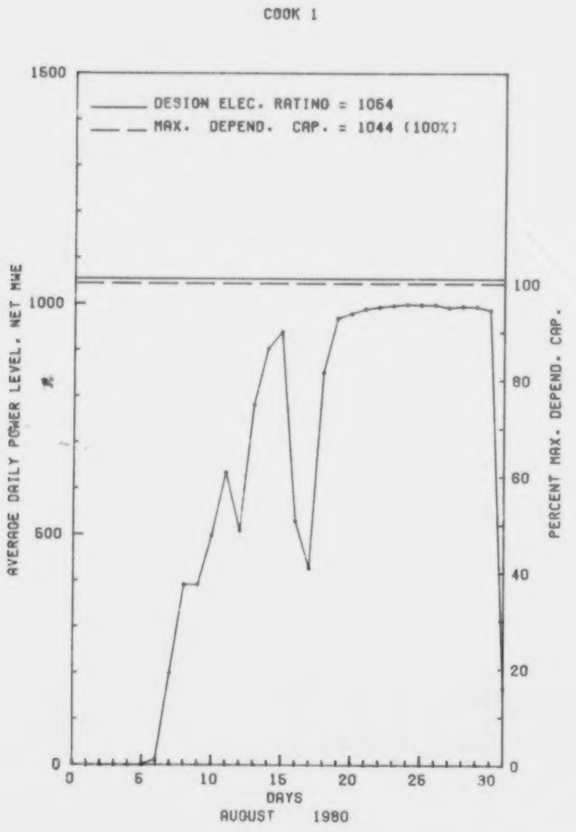
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-14/03L-0	06/12/80	07/10/80	Divider barrier seal deterioration.
80-15/03L-0	07/03/80	07/29/80	While in mode 6 the train "B" centrifugal charging pump failed while running due to a broken pump shaft.
80-16/03L-0	07/20/80	08/19/80	During a surveillance test on the automatic load sequence timing relays, the timer for 1E-MDAFP was found to exceed the maximum allowable time by 2.88 seconds.

REVIEWED BY OMPA LOEB: T. C. Cintula DATE: 10.6.80

REVIEWED BY NRR DL: DATE: 10/6/80

COOK 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-315
2. Utility Contact: W. T. Gillett (616) 465-5901
3. Licensed Thermal Power (MWt): 3250
4. Nameplate Rating (Gross MWe): 1280 x 0.9 = 1152
5. Design Electrical Rating (Net MWe): 1054
6. Maximum Dependable Capacity (Gross MWe): 1080
7. Maximum Dependable Capacity (Net MWe): 1044
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	5,855.0	49,679.0
12. Hours Reactor Was Critical	644.4	3,852.3	36,814.0
13. Reactor Reserve Shutdown Hours	.0	.0	463.0
14. Hours Generator On-Line	574.6	3,774.3	35,857.5
15. Unit Reserve Shutdown Hours	.0	.0	321.0
16. Gross Thermal Energy (MWH)	1,501,561	11,620,492	100,686,797
17. Gross Electrical Energy (MWH)	484,090	3,846,900	33,004,240
18. Net Electrical Energy (MWH)	463,193	3,708,995	31,706,309
19. Unit Service Factor	77.2	64.5	75.4
20. Unit Availability Factor	77.2	64.5	75.4
21. Unit Capacity Factor (MDC Net)	59.6	60.7	66.2
22. Unit Capacity Factor (DER Net)	59.1	60.1	61.8
23. Unit Forced Outage Rate	5.6	10.8	7.2

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
160	05/30/80	S	134.6	C	4		ZZ	ZZZZZZ	THE UNIT WAS REMOVED FOR CYCLE IV-V REFUELING OUTAGE ON 05/30/80. FOLLOWING COMPLETION OF THE OUTAGE AND LOW POWER PHYSICS TESTING THE UNIT WAS RETURNED TO SERVICE ON 08/06/80 AT 1435 HOURS. THE TOTAL LENGTH OF THE OUTAGE WAS 1624.1 HOURS.
161	08/06/80	F	7.1	G	3		ZZ	ZZZZZZ	TURBINE/REACTOR TRIP DUE TO EXTREME HIGH LEVEL IN RIGHT MOISTURE SEPARATOR SHELL DURING STARTUP DUE TO AN ALTERNATE DRAIN VALVE BEING CLOSED.
162	08/07/80	S	1.0	B	1		ZZ	ZZZZZZ	UNIT REMOVED FROM SERVICE TO PERFORM TURBINE OVERSPEED TRIP TESTING. UNIT RETURNED TO SERVICE AND THE POWER ESCALATION TEST PROGRAM COMMENCED.
163	08/16/80	F	6.2	G	3		ZZ	ZZZZZZ	REACTOR/TURBINE TRIP FROM 95% POWER. TRIP WAS DUE TO A FALSE STEAM FLOW/FEEDWATER FLOW MISMATCH COINCIDENT WITH LOW LEVEL IN NO. 3 STEAM GENERATOR CAUSED BY OPERATOR ERROR DURING INSTRUMENT SURVEILLANCE TESTING.
164	08/16/80	F	2.2	B	1		ZZ	ZZZZZZ	TRIPPED TURBINE-GENERATOR MANUALLY AFTER 1.2 HOURS OF OPERATION DUE TO HIGH VIBRATION ON NO. 3 TURBINE BEARING. UNIT RETURNED TO SERVICE AND POWER INCREASED TO 92%.
165	08/31/80	F	18.3	A	3	80-020	ED	ZZZZZZ	REACTOR/TURBINE TRIP DUE TO FAILURE OF INVERTER FOR VITAL A.C. INSTRUMENT BUS CHANNEL IV. UNIT RETURNED TO SERVICE AND POWER INCREASED TO 95%.

SUMMARY

D.C. COOK I ENDED ITS REFUELING OUTAGE ON AUGUST 5TH, AND OPERATED WITH 5 ADDITIONAL OUTAGES LISTED IN DETAIL ABOVE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

COOK 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1. Location (state/county) (nearest population center)	Michigan/Berrien 11 miles S of Benton Harbor
2. Electric Reliability Council	East Central Area Reliability Coordination Agreement
3. Type of Reactor	PWR
4. Power Level (MWT)	3391
5. Design Electrical Rating (MWE Net)	1100
6. Date of Initial Criticality	March 10, 1978
7. Date of Commercial Operation	July 1, 1978
8. Condenser Cooling Method	Once Through
9. Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10. Licensee	AEP Corporation Indiana & Michigan Electric Co.
11. Corporate Address	2 Broadway New York, New York 10004
12. Corporate Contact	John Dolan, VP/Chief Engineer (212) 422 - 4800
13. Architect/Engineer	American Elec. Power Service Corp.
14. Nuclear Steam Supply System	Westinghouse
15. Constructor	Indiana & Michigan Power Co.

Regulatory Information

16. IE Region responsible	III
17. IE Branch Chief	R. Heishman
18. IE Section Chief	D.C. Boyd
19. IE Resident Inspector	E. Swanson (616) 465-6775
20. Licensing Project Manager	S. Miner
21. Docket Number	50-316
22. License Number/Date of Issue	DPR-74, 12/23/77
23. Public Document Room	Maude Preston Palenske Memorial Library 500 Market Street St. Joseph, Michigan 49085

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

Operating routinely.

LAST IE SITE INSPECTION DATE: June 23-27, 1980

INSPECTION REPORT NO: 80-10

SUMMARY

INSPECTION SUMMARY

Inspection on June 23-27, (80-10): Special, announced Performance Appraisal Inspection of the licensee's program and documentation associated with the control of centrifugal Charging/Safety Injection Pumps manufactured by Pacific Pump Division of Dresser Industries. The inspection involved 32 inspector hours onsite by one Performance Appraisal Branch inspector. Of the areas inspected, no items of noncompliance or deviations were identified.

REPORTS RECEIVED FROM LICENSEE

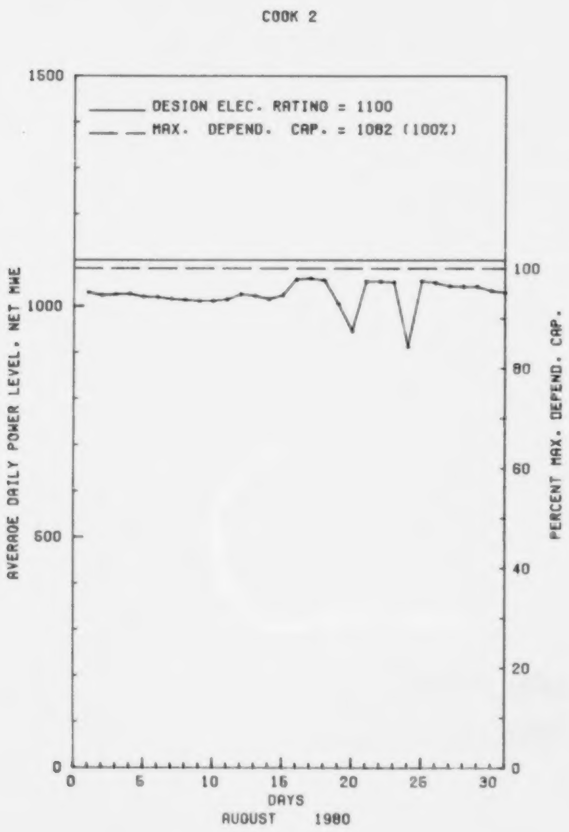
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-28/ 03L-0	07/09/80	07/29/80	During testing of the West -MDAFP during blackout conditions revealed two safety injection relay timers exceeded tolerance.
80-29/ 03L-0	07/21/80	08/19/80	The filter paper motor bound-up stopping the filter paper, rendering R-11 inoperable.

REVIEWED BY OMPA LOEB: T. C. Cintula **DATE:** 10·6·80

REVIEWED BY NRR DL: Sydney Miner **DATE:** 10/6/80

COOK 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-316
2. Utility Contact: W. T. Gillett (616) 465-5901
3. Licensed Thermal Power (MWt): 3391
4. Nameplate Rating (Gross MWe): 1333 x 0.85 = 1133
5. Design Electrical Rating (Net MWe): 1100
6. Maximum Dependable Capacity (Gross MWe): 1118
7. Maximum Dependable Capacity (Net MWe): 1082
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	5,855.0	23,375.0
12. Hours Reactor Was Critical	744.0	5,011.4	16,091.3
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	744.0	4,919.4	15,421.3
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	2,496,594	16,099,570	48,323,950
17. Gross Electrical Energy (MWH)	791,640	5,234,190	15,388,600
18. Net Electrical Energy (MWH)	763,384	5,049,210	14,816,622
19. Unit Service Factor	100.0	84.0	74.1
20. Unit Availability Factor	100.0	84.0	74.1
21. Unit Capacity Factor (MDC Net)	94.8	79.7	68.6
22. Unit Capacity Factor (DER Net)	93.3	78.4	67.9
23. Unit Forced Outage Rate	.0	1.8	9.9

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
81	08/19/80	F	.0	A	5		ZZ	ZZZZZZ	REACTOR POWER REDUCED TO 55% TO REMOVE EAST MAIN FEED PUMP TURBINE FROM SERVICE TO CHECK FEEDPUMP TURBINE CONDENSER FOR TUBE LEAKS. TWO TUBES WERE PLUGGED. REACTOR POWER RETURNED TO 100% 08/20/80.
82	08/24/80	F	.0	A	5		ZZ	ZZZZZZ	REACTOR POWER REDUCED TO 60% TO REMOVE THE EAST MAIN FEED PUMP TURBINE FROM SERVICE TO CHECK FEED PUMP TURBINE CONDENSER FOR TUBE LEAKS. FIVE TUBES WERE PLUGGED. REACTOR POWER RETURNED TO 100% THE SAME DAY.

SUMMARY

D.C. COOK II OPERATED AT APPROXIMATELY 95% POWER FOR THE MONTH, WITH TWO REDUCTIONS TO WORK ON TURBINE CONDENSER TUBE LEAKS.

LEGEND

[1] Type:
F–Forced
S–Scheduled

[2] Reason:
A–Equipment Failure
B–Maintenance or Test
C–Refueling
D–Regulatory Restriction
E–Operator Training & License Examination
F–Administrative
G–Operational Error
H–Other

[3] Method:
1–Manual
2–Manual Scram
3–Automatic Scram
4–Continuations
5–Load Reduction
9–Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

COOPER STATION

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Nebraska/Nemaha 23 miles S of Nebraska City
2.	Electric Reliability Council	Mid-Continent Area Reliability Coordination Agreement
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2381
5.	Design Electrical Rating (MWE Net)	778
6.	Date of Initial Criticality	February 21, 1974
7.	Date of Commercial Operation	July 1, 1974 (50% rating)
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Missouri River

Utility & Contractor Information

10.	Licensee	Nebraska Public Power District
11.	Corporate Address	P. O. Box 499 Columbus, Nebraska 68601
12.	Corporate Contact	J. M. Pilant, Director Licensing and Quality Assurance
13.	Architect/Engineer	Burns and Roe, Inc.
14.	Nuclear Steam Supply System	GE
15.	Constructor	Burns and Roe, Inc.

Regulatory Information

16.	IE Region responsible	IV
17.	IE Branch Chief	G. L. Madsen
18.	IE Section Chief	T. F. Westerman
19.	IE Resident Inspector	R. G. Spangler
20.	Licensing Project Manager	V. L. Rooney
21.	Docket Number	50-298
22.	License Number/Date of Issue	DPR-46, 1/18/74
23.	Public Document Room	Auburn Public Library 118 15th Street Auburn, Nebraska 68305

INSPECTION STATUS

ENFORCEMENT STATUS

THE INSPECTOR FOUND A BREACHED FIRE BARRIER IN THE CABLE SPREADING ROOM. THE BARRIER WAS BREACHED WHILE INSTALLING CONDUIT AS PART OF A DESIGN CHANGE COMPLETED IN 1977. THIS WAS NOT IDENTIFIED BY THE LICENSEE DURING A SUBSEQUENT SURVEILLANCE INSPECTION. THIS IS CONTRARY TO LCO 3.19A AND 3.19B. B.
(8011 INFRACTION)

THE DOOR TO THE HPCI PUMP ROOM WAS FOUND PROPPED OPEN. THIS IS CONTRARY TO ADMINISTRATIVE PROCEDURE 1.6 WHICH REQUIRES THAT ALL FIRE DOORS BE CLOSED.
(8011 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS:

OPERATING AT RATED THERMAL POWER.

LAST IE SITE INSPECTION DATE: August 18 - 21, 1980

INSPECTION REPORT NO: 50-298/80-12

SUMMARY

INSPECTION SUMMARY

Inspection on July 22-26 and August 4-7, 1980 (80-11): Routine, unannounced inspection of followup to previously identified items, Local Power Range Monitoring (LPRM) calibration, core thermal power evaluation, followup on responses to IE Bulletins, followup on LER's, independent inspection effort and the Systematic Assessment of License Performance (SALP) management meeting. Within the six (6) areas inspected, no items of noncompliance were identified in five (5) of the areas. Within the area of independent inspection effort, two (2) items of noncompliance were identified (infraction - failure to control fire doors in accordance with procedures; infraction - failure to maintain fire barrier integrity. As a result of the SALP evaluation no increase in the scope of frequency of inspection effort is planned.

Inspection on August 18-21, 1980 (80-12): Routine, unannounced inspection of followup to previously identified items, housekeeping and cleanliness program, organization and administration, qualification of personnel program, followup on IE Bulletins and followup on LER's. No items of noncompliance or deviation were identified.

REPORTS RECEIVED FROM LICENSEE

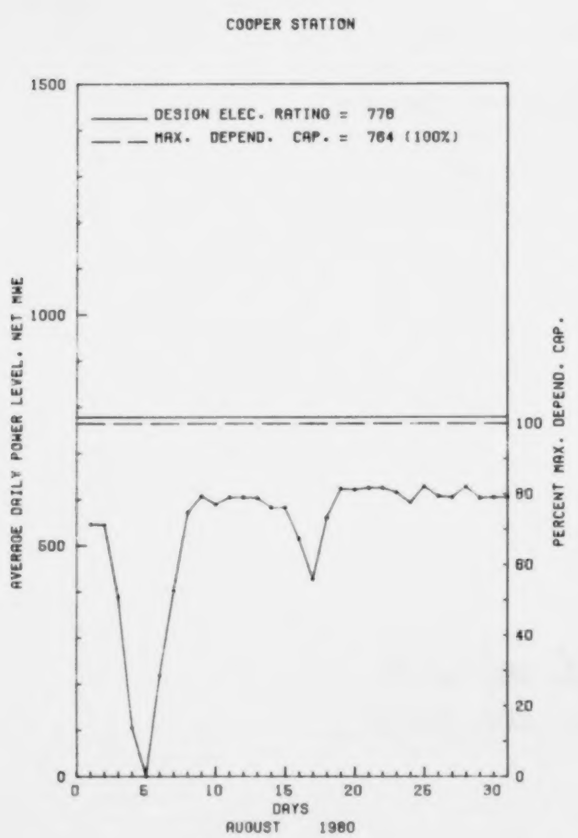
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-25 03L-0	7/16/80	7/25/80	Discharge canal temperature exceeded Technical Specification limit of 103 deg F
80-26 03L-0	7/1/80	7/30/80	Inoperable DWED sump flow recorder
80-27 03L-0	5/8/80	8/8/80	DG #2 connecting rod pin failure
80-28 03L-0	7/13/80	8/11/80	RB vent monitor inoperable
80-29 03L-0	7/23/80	8/12/80	Fire hydrant valve failed
80-30 03L-0	7/13/80	8/21/80	RHR heat exchanger bypass valve failed
80-31 01T-0	8/4/80	8/18/80	Reactor scram on Group 1 isolation
80-32 03L-0	7/25/80	8/22/80	HPCI steamline isolation during surveillance testing

REVIEWED BY OMPA LOEB: John L Crooks DATE: 10/6/80

REVIEWED BY NRR DL: DATE: [illegible]

COOPER STATION

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-298
2. Utility Contact: P. J. Borer (402) 825-3811
3. Licensed Thermal Power (MWt): 2381
4. Nameplate Rating (Gross MWe): 983 x 0.85 = 836
5. Design Electrical Rating (Net MWe): 778
6. Maximum Dependable Capacity (Gross MWe): 787
7. Maximum Dependable Capacity (Net MWe): 764
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): 635
10. Reasons for Restrictions, If Any:
REPLACE TURBINE BLADES & DISCS.

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	5,855.0	54,096.0
12. Hours Reactor Was Critical	730.8	3,544.7	44,782.0
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	698.0	3,450.4	43,926.0
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,498,896	7,379,328	83,562,054
17. Gross Electrical Energy (MWH)	411,942	2,197,923	26,736,771
18. Net Electrical Energy (MWH)	394,934	2,112,149	25,770,426
19. Unit Service Factor	93.8	58.9	81.2
20. Unit Availability Factor	93.8	58.9	81.2
21. Unit Capacity Factor (MDC Net)	69.5	47.2	62.4
22. Unit Capacity Factor (DER Net)	68.2	46.4	61.2
23. Unit Forced Outage Rate	6.2	2.5	4.4

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-09	08/03/80	S	.0	H	5				POWER WAS REDUCED TO ALLOW ADJUSTMENT OF THE CONTROL ROD PATTERN.
80-10	08/04/80	F	46.0	A	3	80-31	CD	ZZZZZZ	AN ELECTRICAL STORM CAUSED A LINE FAULT ON 345 KV DISTRIBUTION SYSTEM AND SUBSEQUENT VOLTAGE TRANSIENTS AFFECTED THE TURBINE CONTROL SYSTEM COMPUTER. MAIN STEAM PRESSURE CONTROL WAS LOST. A GROUP I ISOLATION OCCURRED ON LOW PRESSURE AND THE REACTOR SCRAMMED ON MSIV CLOSURE.

SUMMARY

COOPER STATION OPERATED WITH ONE REDUCTION ON AUGUST 3RD AND ONE OUTAGE WHEN SCRAMMED BY A LINE FAULT CAUSED BY AN ELECTRICAL STORM.

LEGEND

[1] Type:
F–Forced
S–Scheduled

[2] Reason:
A–Equipment Failure
B–Maintenance or Test
C–Refueling
D–Regulatory Restriction
E–Operator Training & License Examination
F–Administrative
G–Operational Error
H–Other

[3] Method:
1–Manual
2–Manual Scram
3–Automatic Scram
4–Continuations
5–Load Reduction
9–Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

CRYSTAL RIVER 3

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Florida/Citrus 7 miles NW of Crystal River
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2452
5.	Design Electrical Rating (MWE Net)	825
6.	Date of Initial Criticality	January 14, 1977
7.	Date of Commercial Operation	March 13, 1977
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Gulf of Mexico

Utility & Contractor Information

10.	Licensee	Florida Power Corporation
11.	Corporate Address	3201 34th Street, South St. Petersburg, Florida 33733
12.	Corporate Contact	J. Hancock
13.	Architect/Engineer	Gilbert Associates, Inc.
14.	Nuclear Steam Supply System	B&W
15.	Constructor	J. A. Jones Construction Co.

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. Lewis
18.	IE Section Chief	R. Martin
19.	IE Resident Inspector	T. F. Stetka
20.	Licensing Project Manager	M. B. Fairtile
21.	Docket Number	50-302
22.	License Number/Date of Issue	DPR-72, 12/3/76
23.	Public Document Room	Crystal River Public Library 668 N.W. First Crystal River, Florida 32639

INSPECTION STATUS

ENFORCEMENT STATUS

INOPERATIVE TAMPER-INDICATING SWITCHES.
(8029 INFRACTION)

UNANNOUNCED DRILLS WERE NOT CONDUCTED AS REQUIRED.
(8029 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

START-UP FROM REFUELING SHUTDOWN.

LAST IE SITE INSPECTION DATE: AUGUST 11-15, 1980

INSPECTION REPORT NO: 50-302/80-30

SUMMARY

INSPECTION SUMMARY

INSPECTION JUNE 30 - JULY 3 (80-26): THIS ANNOUNCED INSPEECTION INVOLVED 22 INSPECTOR-HOURS ON SITE IN THE AREAS OF CONTAINMENT INTEGRATED LEAKAGE RATE TESTING AND SAFETY-RELATED PIPING SUPPORT AND RESTRAINT SYSTEMS. OF THE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION JULY 7-11 (80-29): THE INSPECTION INVOLVED 37 INSPECTOR HOURS ON SITE BY ONE NRC INSPECTOR. THE INSPECTION WAS BEGUN DURING AN OFFSHIFT PERIOD; 10 INSPECTION HOURS WERE ACCOMPLISHED DURING OFFSHIFT PERIODS. AREAS INSPECTED INCLUDED REVIEW OF THE SECURITY PROGRAM AUDIT, SECURITY ORGANIZATION, PROTECTED/VITAL AREA PHYSICAL BARRIERS, LIGHTING, ACCESS CONTROLS (IDENTIFICATION, AUTHORIZATION AND BADGING; SEARCH; AND VITAL AREAS), ALARM STATIONS, DETECTION AIDS, POWER SUPPLY, AND RESPONSE. THE LICENSEE WAS FOUND TO BE IN COMPLIANCE WITH NRC REQUIREMENTS WITHIN THE AREAS EXAMINED DURING THIS INSPECTION EXCEPT FOR THE FOLLOWING TWO ITEMS: 1. DETECTION AIDS - TAMPER INDICATING CAPABILITY COULD NOT BE DEMONSTRATED FOR THE ENTIRE ALARM SYSTEM (INFRACTION); AND 2. RESPONSE - REQUIRED SECURITY EMERGENCY DRILLS WERE NOT CONDUCTED (INFRACTION).

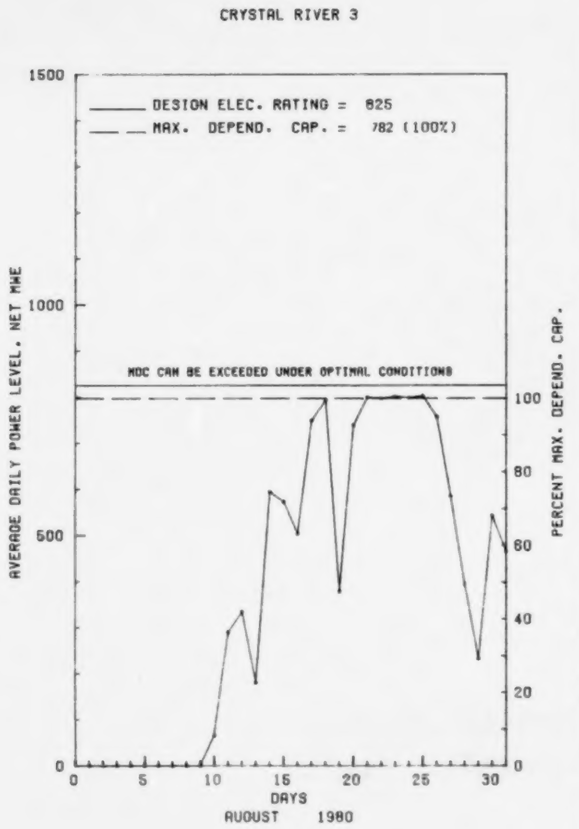
REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-025/ 03L-0	06/09/80	07/07/80	SOURCE RANGE COUNT RATE AMPLIFIER OUT-OF TOLERANCE
80-026/ 03L-0	07/14/80	08/07/80	RCS CHLORIDE AND FLOURIDE CONCENTRATIONS OUT OF TECH SPEC LIMITS
80-027/ 03T	07/23/80	08/07/80	CFS CHECK VALVE FAILED ALLOWING WATER TO ENTER NITROGEN SYSTEM AND BACKFLOW INTO UNRESTRICTED AREA
80-028/ 04T	01/19/80	08/11/80	SR-90 FOUND TO BE TEN TIMES CONTROL VALUE IN GREEN LEAFY VEGETABLE SAMPLE
80-029/ 04L	07/11/80	08/11/80	SAMPLES NOT OBTAINED FOR THE MONTH OF JUNE FROM TEST WELL 4

REVIEWED BY OMPA LOEB: John **DATE:** 10/6/80
REVIEWED BY NRR DL: P B Erickson **DATE:** 10/8/80

CRYSTAL RIVER 3

AVERAGE POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-302
2. Utility Contact: M. W. CULVER (904) 795-6486
3. Licensed Thermal Power (MWt): 2452
4. Nameplate Rating (Gross MWe): 989 × 0.9 = 890
5. Design Electrical Rating (Net MWe): 825
6. Maximum Dependable Capacity (Gross MWe): 821
7. Maximum Dependable Capacity (Net MWe): 782
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	5,855.0	30,431.0
12. Hours Reactor Was Critical	533.1	1,838.6	16,988.8
13. Reactor Reserve Shutdown Hours	.0	.0	1,116.9
14. Hours Generator On-Line	484.2	1,781.6	16,481.9
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	895,784	3,981,711	36,298,622
17. Gross Electrical Energy (MWH)	313,948	1,372,344	12,344,112
18. Net Electrical Energy (MWH)	295,795	1,305,130	11,696,770
19. Unit Service Factor	65.1	30.4	54.2
20. Unit Availability Factor	65.1	30.4	54.2
21. Unit Capacity Factor (MDC Net)	50.8	28.5	49.2
22. Unit Capacity Factor (DER Net)	48.2	27.0	46.6
23. Unit Forced Outage Rate	5.5	14.9	31.4

24. Shutdowns Scheduled Over Next 6 Months (Type, Date and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-09	08/01/80	S	231.5	C	4		RC	FUELXX	CONCLUSION OF REFUELING OUTAGE.
80-10	08/12/80	F	11.0	A	3		CH	PUMPXX	REACTOR TRIPPED DUE TO FWP-2B TRIPPING ON OVERSPEED.
80-11	08/16/80	F	16.9	A	5		CH	PUMPXX	REDUCED POWER TO APPROXIMATELY 50% FP TO REPAIR A STEAM LEAK ON FWP-2B.
80-12	08/19/80	F	6.9	G	3		CH	HTEXCH	A NEW INSTRUMENT TECHNICIAN MISTAKENLY ADJUSTED THE LEVEL SWITCH ON FWHE 2B INSTEAD OF THE SWITCH ON "D" CONDENSER WATERBOX. THIS CAUSED THE TURBINE TO TRIP, WHICH TRIPPED THE REACTOR.
80-13	08/26/80	F	70.5	H	5		HC	HTEXCH	REDUCED POWER TO VARYING BETWEEN 45% AND 80% FP DUE TO WATERBOX CLEANING AND HIGH CONDENSER DELTA T.
80-14	08/29/80	F	10.4	A	3		HA	VALVEX	THE REFERENCE ROD FOR THE LINER VOLTAGE DIFFERENTIAL TRANSFORMER ON GV-3 (GOVERNOR VALVE) FAILED GIVING NO POSITIONAL INDICATION. GV-3 WENT FULL OPEN, TURBINE HEADER PRESSURE DECREASED, AND FW INCREASED. THEN THE VALVE CLOSED, HEADER PRESSURE RECOVERED, FW DROPPED AND THE RX TRIPPED ON HIGH PRESSURE.

SUMMARY

CRYSTAL RIVER 3 CONCLUDED REFUELING OUTAGE ON THE 10TH, AND OPERATED FOR THE BALANCE OF THE MONTH WITH THREE OUTAGES AND TWO POWER REDUCTIONS FOR EQUIPMENT FAILURE AND AN OPERATIONAL ERROR.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

DAVIS-BESSE 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Ohio/Ottawa 21 miles E of Toledo
2.	Electric Reliability Council	East Central Area Reliability Coordination Agreement
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2772
5.	Design Electrical Rating (MWE Net)	906
6.	Date of Initial Criticality	August 12, 1977
7.	Date of Commercial Operation	11/21/77 (25%); 12/19/77 (40%) 01/23/78 (75%); 07/31/78 (100%)
8.	Condenser Cooling Method	Cooling Tower
9.	Condenser Cooling Water Source	Lake Erie

Utility & Contractor Information

10.	Licensee	Toledo Edison Co.
11.	Corporate Address	Edison Plaza, 300 Madison Avenue Toledo, Ohio 43652
12.	Corporate Contact	R. P. Crouse, V. P., Nuclear (419)259-5221
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	B&W
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	R. F. Warnick
19.	IE Resident Inspector	L. Reyes (419) 898-2775
20.	Licensing Project Manager	D. Garner
21.	Docket Number	50-346
22.	License Number/Date of Issue	NPF-3, 4/22/77
23.	Public Document Room	Ida Rupp Public Library 310 Madison Street Port Clinton, OH 43452

INSPECTION STATUS

ENFORCEMENT STATUS

FAILURE TO ADEQUATELY AND EFFECTIVELY CONTROL ACCESS IN ACCORDANCE WITH A SECURITY PLAN COMMITMENT.
(8018 INFRACTION)

THE LICENSEE FAILED TO ASSESS THE ADEQUACY OF THE SEARCH REQUIRED FOR AUTHORIZATION IN ACCORDANCE WITH SECURITY PLAN COMMITMENTS.
(8018 INFRACTION)

FAILURE TO PROVIDE COMMUNICATION CAPABILITIES IN ACCORDANCE WITH A SECURITY PLAN COMMITMENT.
(8018 INFRACTION)

TECHNICAL SPECIFICATION 4.6.1.2 REQUIRES THAT THE CONTAINMENT LEAKRATE SHALL BE DEMONSTRATED USING THE METHODS AND PROVISIONS OF ANSI N45.4-1972. ANSI N45.4-1972 REQUIRES THAT A DATED LOG OF EVENTS AND PERTINENT OBSERVATIONS SHALL BE MAINTAINED DURING THE TEST. PROCEDURE ST 50.61 "CONTAINMENT INTEGRATED LEAKRATE TEST," ALSO REQUIRES THAT A CONTINUOUS SEQUENTIAL LOG OF EVENTS OCCURRING DURING THE PERFORMANCE OF THE TEST BE MAINTAINED. CONTRARY TO THE ABOVE, A LOG OF EVENTS WAS NOT MAINTAINED FOR THE DURATION OF THE CONTAINMENT LEAKRATE TEST PERFORMED ON JUNE 9 THROUGH JUNE 13, 1980.
(8019 DEFICIENCY)

TECHNICAL SPECIFICATION 6.8.1 REQUIRES THAT WRITTEN PROCEDURES RECOMMENDED IN APPENDIX "A" OF REGULATORY GUIDE 1.33, NOVEMBER 1972 SHALL BE IMPLEMENTED. APPENDIX "A" OF REGULATORY GUIDE 1.33, NOVEMBER 1972, INCLUDES ADMINISTRATIVE PROCEDURES. ADMINISTRATIVE PROCEDURE AD1839.02 "OPERATION AND CONTROL OF LOCKED VALVES" REQUIRES THAT WHEN LOCKED VALVES ARE REPOSITIONED THE APPROPRIATE LOG ENTRIES ARE COMPLETED IN THE LOCKED VALVE LOG. AF599 AND AF608 ARE LOCKED VALVES. CONTRARY TO THE ABOVE, ON JUNE 17, 1980 AT APPROXIMATELY 7:00 A.M. THE INSPECTOR DETERMINED THAT VALVES AF599 AND AF608 WERE IN THE OPEN POSITION AND THE LOCKED VALVE LOG INDICATED THEY WERE CLOSED.
(8019 DEFICIENCY)

TECHNICAL SPECIFICATION 6.8.1 REQUIRES THAT PROCEDURES COVERING THE FIRE PROTECTION PROGRAM SHALL BE IMPLEMENTED. PROCEDURE ST 5016.07 REQUIRES THAT THE INSPECTOR OF FIRE EXTINGUISHERS SHALL SIGN THE INSPECTION TAG OF THE FIRE EXTINGUISHER. THE INITIALED INSPECTION TAG SIGNIFIES COMPLIANCE WITH THE MONTHLY INSPECTION PROCEDURE. CONTRARY TO THE ABOVE, ON JULY 6 AND 20, 1980, THE NRC INSPECTOR OBSERVED THAT SEVERAL FIRE EXTINGUISHERS DID NOT HAVE THEIR INSPECTION TAGS SIGNED FOR THE INSPECTION DURING THE MONTH OF JUNE. SEVERAL OTHER EXTINGUISHERS HAD THEIR INSPECTION TAGS MISSING.
(8019 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENTS

A total of 133 fuel assembly holddown springs were replaced. Of those replaced, 18 were broken. Forty-four new assemblies were loaded for cycle 2. (80-19)

PLANT STATUS

The refueling/maintenance outage continued during the month.

LAST IE SITE INSPECTION DATE: July 21-25, 1930

INSPECTION REPORT NO: 80-24

SUMMARY

INSPECTION SUMMARY

Inspection on June 16-20, 23-27, 30; July 1-3 and 7-11, (80-19): Routine, unannounced inspection of followup on previous Inspection Findings, Containment Integrated Leak Rate Test, TI2515/36, Monthly Surveillance Observation, Monthly Maintenance Observation, IE Bulletin Followup, Onsite Review Committee, Inspection During Long-Term Shutdown, Licensee Event Reports Followup, and Meeting with the Public and Local Officials. The inspection involved 84 inspector-hours during regular shift and 30 inspector-hours during offshifts by the Resident Inspector. Of the 10 areas inspected, no items of noncompliance or deviations were identified in eight areas; three apparent items of noncompliance were identified in the other two areas (failure to keep a log of events during CILRT; failure to document inspection of fire extinguishers; and failure to properly log the position of locked valves).

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-55/ 03L-0	07/09/80	08/08/80	Position indication for ECS room vent. exhaust damper lost.
80-56/ 03L-0	07/09/80	08/07/80	Loss of instrument bus Y-3.
80-57/ 03L-0	07/10/80	08/07/80	Decay heat flow rate less than 2800 gpm while in Mode 6.
80-58/ 03L-0	07/24/80	08/12/80	Loss of decay heat flow.

Inspection on July 22-24, 31; and August 1, (80-22): Inspection of pipe whip restraint modification at Hot Leg risers; licensee implementation of IEB 79-02, IEB 79-04, and IEB 79-14; snubber installations. The inspection involved 45 inspector-hours onsite, at the licensee's A-E, and corporate office by two NRC inspectors. No items of noncompliance or deviations were identified.

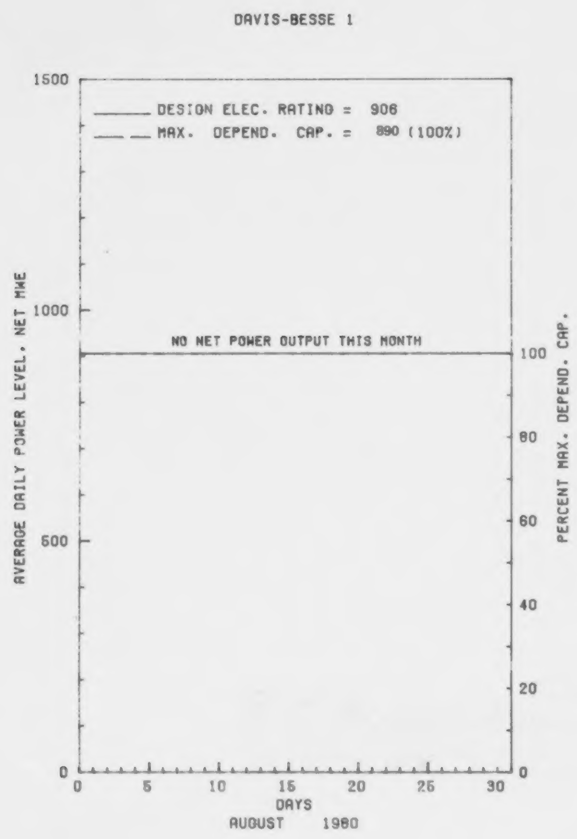
Inspection on July 21-25, (80-24): Routine, unannounced inspection of: Environmental Protection, including evaluation of reportable occurrences 78-072-04T-0, NP-39-78-03, NP-09-80-01, and NP-11-80-01; the licensee's program for quality control of analytical measurements; program management and implementation; review of program results; inspection of proper installation and operability of selected sampling stations; Emergency Planning - review of coordination with offsite support agencies; emergency tests and drills; and Confirmatory Measurements - program for quality assurance and quality control of analytical measurements. The inspection involved 74 inspector-hours onstie by three NRC inspectors. Of the seven areas inspected, no apparent items of noncompliance or deviations were identified in six areas; four apparent items of noncompliance were found in one area; (delta T in excess of Technical Specification limits; failure to report exceeding Technical Specification limiting condition of operation; failure to perform required sulfate monitoring at a required interval; and failure to perform required ichthyoplankton monitoring at a required interval).

REVIEWED BY OMPA LOEB: John L Crook **DATE:** 10/6/80

REVIEWED BY NRR DL: David J Garner **DATE:** 10/8/80

DAVIS-BESSE 1

AVERAGE POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-346
2. Utility Contact: BILAL SARSOUR (419) 259-5000 X236
3. Licensed Thermal Power (MWt): 2772
4. Nameplate Rating (Gross MWe): 1069 x 0.9 = 962
5. Design Electrical Rating (Net MWe): 906
6. Maximum Dependable Capacity (Gross MWe): 934
7. Maximum Dependable Capacity (Net MWe): 890
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	5,855.0	26,380.0
12. Hours Reactor Was Critical	.0	2,078.2	13,042.4
13. Reactor Reserve Shutdown Hours	.0	.0	2,875.8
14. Hours Generator On-Line	.0	2,008.7	11,883.4
15. Unit Reserve Shutdown Hours	.0	.0	1,728.2
16. Gross Thermal Energy (MWH)	0	4,687,304	24,886,811
17. Gross Electrical Energy (MWH)	0	1,583,559	8,307,070
18. Net Electrical Energy (MWH)	0	1,483,787	7,654,365
19. Unit Service Factor	.0	34.3	45.8
20. Unit Availability Factor	.0	34.3	52.9
21. Unit Capacity Factor (MDC Net)	.0	28.5	34.8
22. Unit Capacity Factor (DER Net)	.0	28.0	34.2
23. Unit Forced Outage Rate	.0	14.3	25.6

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 09/15/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
4	04/07/80	S	744.0	C	4				THE UNIT OUTAGE WHICH BEGAN ON APRIL 7, 1980, WAS STILL IN PROGRESS THROUGH THE END OF AUGUST, 1980.

SUMMARY

THE UNIT REMAINS SHUTDOWN FOR SYSTEMS MODIFICATIONS, INCLUDING REQUIREMENTS FOR NRC BULLETINS 79-02 AND 79-14.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

DRESDEN 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Illinois/Grundy 9 miles E of Morris
2.	Electric Reliability Council	Mid-America Interpool Network
3.	Type of Reactor	BWR
4.	Power Level (MWT)	700
5.	Design Electrical Rating (MWE Net)	200
6.	Date of Initial Criticality	October 15, 1959
7.	Date of Commercial Operation	July 4, 1960
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Kankakee River

Utility & Contractor Information

10.	Licensee	Commonwealth Edison Co.
11.	Corporate Address	P. O. Box 767 Chicago, Illinois 60690
12.	Corporate Contact	C. Reed
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	GE
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	R. Spessard
19.	IE Resident Inspector	J. L. Barker (815)-942-9267
20.	Licensing Project Manager	Paul W. O'Connor
21.	Docket Number	50-010
22.	License Number/Date of Issue	DPR-2 issued 9/28/59
23.	Public Document Room	Morris Public Library 604 Liberty Street Morris, Illinois 60451

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

+ Unit 1 shutdown on October 31, 1978 for an extended outage to upgrade ECCS, chemical cleaning and refueling. Outage expected to last through Mid-1986.

+LAST IE SITE INSPECTION DATE: July 7 through August 1, 1980

INSPECTION REPORT NO: 80-13

SUMMARY

INSPECTION SUMMARY

+ Inspection on July 7 through August 1, (80-13): Routine, unannounced inspection of monthly surveillance observation; licensee event reports followup; IE Bulletin followup; training; requalification training; onsite review committee; inspection during long-term shutdown. The inspection involved 238 inspector-hours onsite by 3 NRC inspectors including 112 inspector-hours onsite during off-shifts. Of the nine areas inspected, there were no items of noncompliance identified in eight areas. There was one apparent item of noncompliance (licensed operators failed to keep themselves cognizant of design changes, facility license changes, and procedure changes) identified in one area.

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
None.			

REVIEWED BY OMPA LOEB: T. C. Cintula DATE: 10.7.80

REVIEWED BY NRR DL: DATE: 10/8/80

DRESDEN 1

AVERAGE DAILY POWER LEVEL (MWe)

OPERATING STATUS

1. Docket Number: 50-010
2. Utility Data Prepared By: S. A. Cieszkiewics (155) 942-2920
3. Licensed Thermal Power (MWt): 700
4. Nameplate Rating (Gross MWe): 245.5 x 0.85 = 209
5. Design Electrical Rating (Net MWe): 200
6. Maximum Dependable Capacity (Gross MWe): 0
7. Maximum Dependable Capacity (Net MWe): 0
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons: None
9. Power Level To Which Restricted, If Any (Net MWe): None
10. Reasons For Restrictions, If Any:

SHUTDOWN INDEFINITELY

	This Month	Y	ulative
11. Hours In Reporting Period			
12. Number Of Hours Reactor Was Critical			
13. Reactor Reserve Shutdown Hour			
14. Hours Generator On-			
15. Unit Reserve Shutd rs			
16. Gross Thermal Energy Generated (MWH)			
17. Gross Electrical Energy Generated (MWH)			
18. Net Electrical Energy Generated (MWH)			
19. Unit Service Factor			
20. Unit Availability Factor			
21. Unit Capacity Factor (Using MDC Net) 1/			
22. Unit Capacity Factor (Using DER Net) 1/			
23. Unit Forced Outage Rate			

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): None
25. If Shut Down At End Of Report Period, Estimated Date of Startup: 06/86

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
SEE SUMMARY BELOW									

SUMMARY

DRESDEN 1 IS SHUTDOWN UNTIL APPROXIMATELY JUNE, 1986 FOR THE FOLLOWING REASONS:

1) HPCI INSTALLATION FOR ECCS UPGRADING
2) EQUIPMENT UPGRADE TO IEEE 279
3) MAJOR INSERVICE INSPECTION
4) CHEMICAL CLEANING
5) REFUEL

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

NRC POWER RESTRICTIONS: NONE

DRESDEN 2

FACILITY DATA

Facility Description

1. Location (state/countv) (nearest population center)	Illinois/Grundy 9 miles E of Morris
2. Electric Reliability Council	Mid-America Interpool Network
3. Type of Reactor	BWR
4. Power Level (MWT)	2527
5. Design Electrical Rating (MWE Net)	794
6. Date of Initial Criticality	January 7, 1970
7. Date of Commercial Operation	JULY, 1970
8. Condenser Cooling Method	Cooling Lake
9. Condenser Cooling Water Source	Kankakee and Des Plaines Rivers

Utility & Contractor Information

10. Licensee	Commonwealth Edison Co.
11. Corporate Address	P. O. Box 767 Chicago, Illinois 60690
12. Corporate Contact	C. Reed
13. Architect/Engineer	Sargent & Lundy
14. Nuclear Steam Supply System	GE
15. Constructor	United Engineers & Constructors, Inc.

Regulatory Information

16. IE Region responsible	III
17. IE Branch Chief	R. Heishman
18. IE Section Chief	R. Spessard
19. IE Resident Inspector	J. L. Barker (815)-942-9267
20. Licensing Project Manager	Paul W. O'Connor
21. Docket Number	50-237
22. License Number/Date of Issue	DPR-19 issued 12/22/69
23. Public Document Room	Morris Public Library 604 Liberty Street Morris, Illinois 60451

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

Unit operating routinely.

LAST IE SITE INSPECTION DATE: July 7 through August 1, 1980

INSPECTION REPORT NO: 80-14

SUMMARY

INSPECTION SUMMARY

Inspection on July 7 through August 1, (80-14): Routine, unannounced inspection of operational safety verification; monthly surveillance observation; licensee event reports followup; IE Bulletin followup; training; requalification training; onsite review committee; inspection during long-term shutdown; and IE Bulletin 80-17 requirements. The inspection involved 238 inspector-hours onsite by 8 NRC inspectors including 112 inspector-hours onsite during off-shifts. Of the nine areas inspected, there were no items of noncompliance identified in eight areas. There was one apparent items of noncompliance (licensed operators failed to keep themselves cognizant of design changes, facility license changes, and procedure changes) identified in one area.

REPORTS RECEIVED FROM LICENSEE

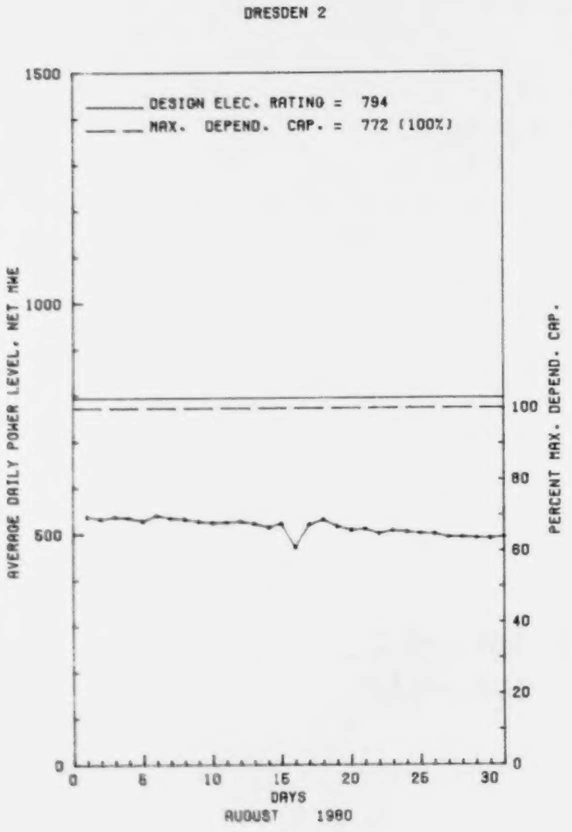
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-24/ 03L-0	07/21/80	08/15/80	During normal operations, drywell oxygen concentration indication failed downscale.
80-25/ 03L-0	07/21/80	08/08/80	While performing isolation condenser condensate line hi-flow calibration, DPIS 2-1349 B tripped at 34 inches H20.
80-26/ 03L-0	07/25/80	08/15/80	While checking operating pressure of drywell high pressure ECCS switch 1629B, it was found that pressure switch had been valved out.
80-27/ 03L-0	07/25/80	08/20/80	During power operation water was found in lube oil in one of diesel generator unit and the diesel was declared inoperable.
80-28/ 03L-0	07/27/80	08/15/80	CRD C-11 was determined to be uncoupled and was declared to be inoperable.

REVIEWED BY OMPA LOEB: John L. Crooks DATE: 10/6/80

REVIEWED BY NRR DL: S. J. Nowicki DATE: 10/8/80

DRESDEN 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-237
2. Utility Contact: S. A. CIESZKIEWICZ (815) 942-2920 X489
3. Licensed Thermal Power (MWt): 2527
4. Nameplate Rating (Gross MWe): 920 x 0.9 = 828
5. Design Electrical Rating (Net MWe): 794
6. Maximum Dependable Capacity (Gross MWe): 812
7. Maximum Dependable Capacity (Net MWe): 772
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	5,855.0	90,311.0
12. Hours Reactor Was Critical	744.0	5,667.3	70,404.0
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	744.0	5,601.5	66,864.2
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,271,029	12,163,903	133,805,735
17. Gross Electrical Energy (MWH)	385,429	3,815,950	42,822,571
18. Net Electrical Energy (MWH)	359,697	3,610,147	40,498,259
19. Unit Service Factor	100.0	95.7	74.0
20. Unit Availability Factor	100.0	95.7	74.0
21. Unit Capacity Factor (MDC Net)	62.6	79.9	58.1
22. Unit Capacity Factor (DER Net)	60.9	77.7	56.5
23. Unit Forced Outage Rate	.0	.8	12.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
REFUELING, DECEMBER 31, 1980, APPROXIMATELY 18 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	NONE								

SUMMARY

DRESDEN 2 OPERATED NORMALLY FOR AUGUST, WITH NO REDUCTIONS OR OUTAGES.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

DRESDEN 3

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Illinois/Grundy 9 miles E of Morris
2.	Electric Reliability Council	Mid-America Interpool Network
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2527
5.	Design Electrical Rating (MWE Net)	794
6.	Date of Initial Criticality	January 31, 1971
7.	Date of Commercial Operation	November 16, 1971
8.	Condenser Cooling Method	Cooling Lake
9.	Condenser Cooling Water Source	Kankakee and Des Plaines Rivers

Utility & Contractor Information

10.	Licensee	Commonwealth Edison Co.
11.	Corporate Address	P. O. Box 767 Chicago, Illinois 60690
12.	Corporate Contact	Cordell Reed
13.	Architect/Engineer	Sargent & Lundy
14.	Nuclear Steam Supply System	GE
15.	Constructor	United Engineers & Constructors, Inc.

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	R. Spessard
19.	IE Resident Inspector	T. Tongue 815-942-9267
20.	Licensing Project Manager	R. Bevan
21.	Docket Number	50-249
22.	License Number/Date of Issue	DPR-25 issued 1/12/71
23.	Public Document Room	Morris Public Library 604 Liberty Street Morris, Illinois 60451

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

Unit operating routinely.

LAST IE SITE INSPECTION DATE: July 7 through August 1, 1980

INSPECTION REPORT NO: 80-18

SUMMARY

INSPECTION SUMMARY

Inspection on July 7 through August 1, (80-18): Routine, unannounced inspection of operational safety verification; monthly surveillance observation; licensee event reports followup; IE Bulletin followup; training; requalification training; onsite review committee; inspection during long-term shutdown; and IE Bulletin 80-17 requirements. The inspection involved 238 inspector-hours onsite by 8 NRC inspectors including 112 inspector-hours onsite during off-shifts. Of the nine areas inspected, there were no items of noncompliance identified in eight areas. There was one apparent items of noncompliance (licensed operators failed to keep themselves cognizant of design changes, facility license changes, and procedure changes) identified in one area.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

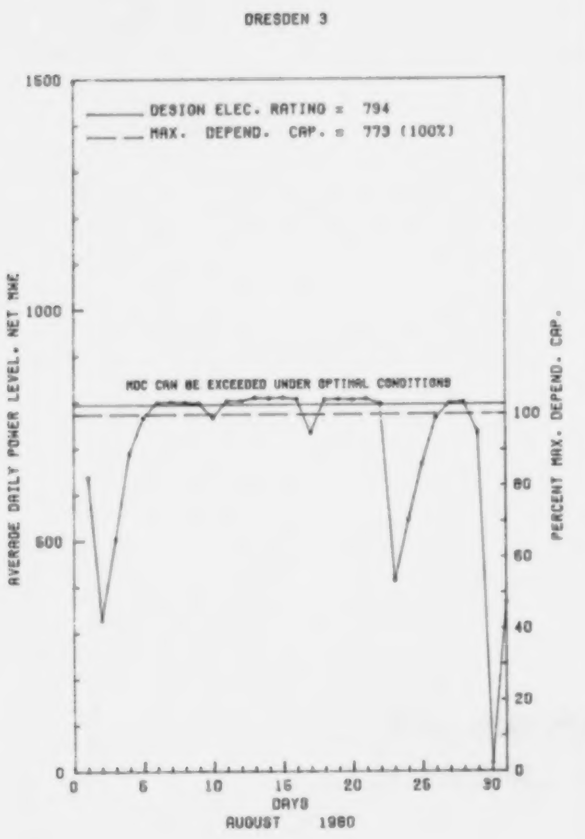
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-28/ 03L-0	07/18/80	08/14/80	Solenoid valve for torus to reactor building vacuum breaker 3-1601-20A inoperative, preventing the valve from opening.
80-29/ 03L-0	07/28/80	08/18/80	Water was leaking from weld on an instrument line off Rx. water cleanup system main header.
80-30/ 03L-0	07/19/80	08/05/80	MSIV closure times exceeded T.S. limits on MSIVs 3-203-2B and 2D.
80-31/ 01L-0	07/19/80	07/28/80	Following a manual scram, an ultrasonic test of the west scram discharge volume revealed the 4 inch pipe was 80% full after all level alarms cleared on instrumented scram volume.
80-32/ 03L-0	07/24/80	08/21/80	While performing APRM rod block and scram functional tests, APRM 3 failed to initiate a rod block.
80-33/ 03L-0	07/24/80	08/18/80	A 1/4 inch through wall leak was discovered in 3A CCWS pump casing.

REVIEWED BY OMPA LOEB: John L. Crooks **DATE:** 10/6/80

REVIEWED BY NRR DL: RBBevan **DATE:** 10/8/80

DRESDEN 3

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-249
2. Utility Contact: S. A. CIESZKIEWICZ (815) 942-2920 X489
3. Licensed Thermal Power (MWt): 2527
4. Nameplate Rating (Gross MWe): 920 x 0.9 = 828
5. Design Electrical Rating (Net MWe): 794
6. Maximum Dependable Capacity (Gross MWe): 812
7. Maximum Dependable Capacity (Net MWe): 773
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	5,855.0	79,896.0
12. Hours Reactor Was Critical	716.7	3,626.4	59,391.8
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	704.2	3,535.4	56,860.0
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,627,689	7,695,766	109,967,434
17. Gross Electrical Energy (MWH)	516,920	2,453,398	35,883,072
18. Net Electrical Energy (MWH)	490,253	2,317,168	34,010,451
19. Unit Service Factor	94.7	60.4	71.2
20. Unit Availability Factor	94.7	60.4	71.2
21. Unit Capacity Factor (MDC Net)	85.2	51.2	55.1
22. Unit Capacity Factor (DER Net)	83.0	49.8	53.6
23. Unit Forced Outage Rate	5.3	3.2	15.8

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
11	08/01/80	F	12.8	A	*				RX SCRAM - LOW REACTOR WATER LEVEL (APPEARS 3"B" FEEDWATER REGULATOR VALVE WENT CLOSED).
12	08/29/80	F	15.8	A	3				RX SCRAM - LOW REACTOR WATER LEVEL (APPEARS 3"C" REACTOR FEED PUMP MINIMUM FLOW VALVE DRIFTED OPEN).
13	08/30/80	F	11.2	H	1				TURBINE TRIPPED ON "C" MOISTURE SEPARATOR HI HI WHILE RESETTING HEATERS. RX SCRAMMED ON HI NEUTRON FLUX.

SUMMARY

DRESDEN 3 OPERATED AT OR NEAR FULL POWER FOR THE MONTH, WITH THREE OUTAGES RELATED TO EQUIPMENT FAILURE ON THE 1ST, 29TH AND 30TH.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

NRC POWER RESTRICTIONS: NONE

DUANE ARNOLD

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Iowa/Linn 8 miles NW of Cedar Rapids
2.	Electric Reliability Council	Mid-Continent Area Reliability Coordination Agreement
3.	Type of Reactor	BWR
4.	Power Level (MWT)	1658
5.	Design Electrical Rating (MWE Net)	538
6.	Date of Initial Criticality	March 23, 1974
7.	Date of Commercial Operation	February 1, 1975
8.	Condenser Cooling Method	Cooling Towers
9.	Condenser Cooling Water Source	Cedar Rapids River Circulating Water System

Utility & Contractor Information

10.	Licensee	Iowa Electric Light and Power
11.	Corporate Address	IE Towers, P. O. Box 351 Cedar Rapids, Iowa 52406
12.	Corporate Contact	D. Arnold, President
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	GE
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	W. S. LITTLE
19.	IE Resident Inspector	W. F. CHRISTIANSON
20.	Licensing Project Manager	K. Eccleston
21.	Docket Number	50-331
22.	License Number/Date of Issue	DPR-49 issued 2/22/74
23.	Public Document Room	Reference Service Cedar Rapids Public Library 428 Third Avenue, S.E. Cedar Rapids, Iowa 52401

INSPECTION STATUS

ENFORCEMENT STATUS

10 CFR 50, APPENDIX B, CRITERIA VI "DOCUMENT CONTROL" STATES, IN PART, "MEASURES SHALL BE ESTABLISHED TO CONTROL THE ISSUANCE OF...PROCEDURES...INCLUDING CHANGES THERETO...THESE MEASURES SHALL ASSURE THAT DOCUMENTS, INCLUDING CHANGES...ARE DISTRIBUTED TO AND USED AT THE LOCATION WHERE THE PRESCRIBED ACTIVITY IS PERFORMED." IOWA ELECTRIC'S QUALITY ASSURANCE DIRECTIVE NO. 1306.1 "DOCUMENT CONTROL" STATES IN PART, "CONTROL SHALL ASSURE THAT DOCUMENTS...ARE DISTRIBUTED TO AND USED AT THE LOCATION OF THE PRESCRIBED ACTIVITY." CONTRARY TO THE ABOVE, REVISION 0, DATED MAY 1, 1977, OF THE VALVE LINEUP SHEET FOR OI-53 "STANDY LIQUID CONTROL SYSTEM" WAS PERFORMED ON APRIL 9, 1980, WITHOUT THE REQUIREMENTS OF A DESIGN CHANGE (DCF 2469) DATED MARCH 19, 1980 BEING INCLUDED. THIS RESULTED IN A VALVE BEING LEFT UNLOCKED WHICH SHOULD HAVE BEEN LOCKED.
(8010 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

Unit operating routinely.

LAST IE SITE INSPECTION DATE: August 4-6, 1980

INSPECTION REPORT NO: 80-12

SUMMARY

INSPECTION SUMMARY

Inspection on August 4-6, (80-12): Routine, unannounced inspection of previously identified items; control rod exercise and scram time tests; LPRM calibrations; reactivity anomaly determination; and core power distribution limits. The inspection involved 28 inspector-hours onsite by two NRC inspectors including zero inspector-hours onsite during off-shifts. The inspectors were accompanied by one NRC inspector-in-training. No items of noncompliance or deviations were identified.

REPORTS RECEIVED FROM LICENSEE

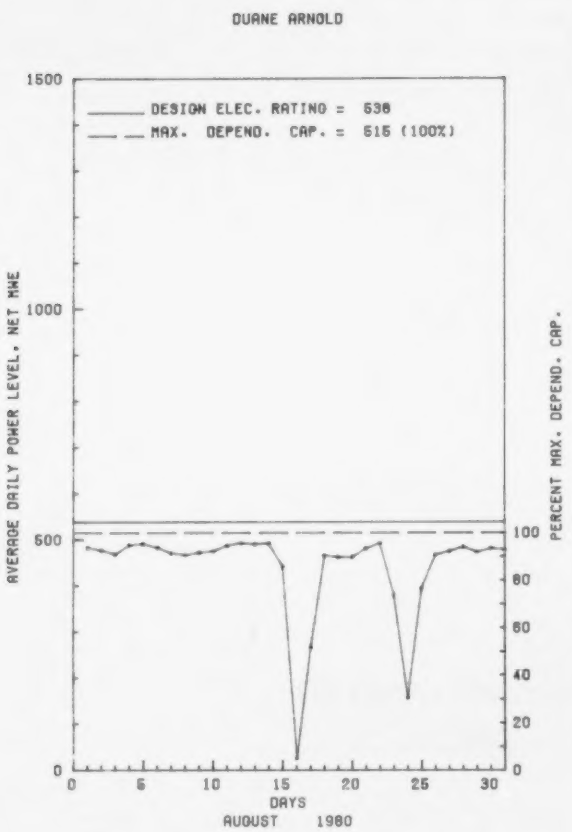
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
	06/10/80	07/10/80	Special Report on diesel fire pump.

REVIEWED BY OMPA LOEB: John L Crooks **DATE:** 10/6/80

REVIEWED BY NRR DL: Ken Eccleston **DATE:** 10/8/80

DUANE ARNOLD

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-331
2. Utility Contact: J. Van Sickel (319) 851-5611
3. Licensed Thermal Power (MWt): 1658
4. Nameplate Rating (Gross MWe): 663 x 0.9 = 597
5. Design Electrical Rating (Net MWe): 538
6. Maximum Dependable Capacity (Gross MWe): 545
7. Maximum Dependable Capacity (Net MWe): 515
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	5,855.0	48,935.0
12. Hours Reactor Was Critical	729.1	3,922.9	34,487.0
13. Reactor Reserve Shutdown Hours	.0	.0	0
14. Hours Generator On-Line	719.0	3,814.5	33,633.6
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,066,752	4,951,872	42,081,432
17. Gross Electrical Energy (MWH)	348,310	1,653,809	14,078,175
18. Net Electrical Energy (MWH)	327,278	1,548,540	13,163,604
19. Unit Service Factor	96.6	65.1	68.7
20. Unit Availability Factor	96.6	65.1	68.7
21. Unit Capacity Factor (MDC Net)	85.4	51.4	52.2
22. Unit Capacity Factor (DER Net)	81.8	49.2	50.0
23. Unit Forced Outage Rate	3.4	7.9	19.6

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
REFUELING, MARCH 7, 1981, - 9 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
14	08/16/80	F	19.9	A	1				SHUTDOWN TO REPAIR NITROGEN LEAK ON FEEDWATER CHECK VALVE.
15	08/23/80	S	.0	H	5				POWER WAS REDUCED IN ORDER TO PERFORM A CONTROL ROD SEQUENCE EXCHANGE.
16	08/24/80	F	5.1	B	1				POWER WAS REDUCED AND THE GENERATOR TAKEN OFF LINE IN ORDER TO PERFORM TURBINE OVERSPEED TRIP TESTING.

SUMMARY

DUANE ARNOLD OPERATED AT NEAR FULL POWER WITH THE EXCEPTION OF TWO OUTAGES AND ONE REDUCTION FOR EQUIPMENT FAILURE, TESTING AND MAINTENANCE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

FARLEY 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Alabama/Houston 24 miles SE of Dothan
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2652
5.	Design Electrical Rating (MWE Net)	829
6.	Date of Initial Criticality	August 9, 1977
7.	Date of Commercial Operation	December 1, 1977
8.	Condenser Cooling Method	Once Through With Forced Draft Cooling Towers
9.	Condenser Cooling Water Source	Chattahoochee River

Utility & Contractor Information

10.	Licensee	Alabama Power Company
11.	Corporate Address	600 North 18th Street Birmingham, Alabama
12.	Corporate Contact	F. L. Clayton, Jr. Senior Vice President (205-323-5341)
13.	Architect/Engineer	Bechtel/SSI
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Daniel

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. Lewis
18.	IE Section Chief	R. D. Martin
19.	IE Resident Inspector	W. H. Bradford (205-899-3386)
20.	Licensing Project Manager	E. A. Reeves
21.	Docket Number	50-348
22.	License Number/Date of Issue	NPF-2, 6/25/77
23.	Public Document Room	G. S. Houston Memorial Library 212 W. Berdeshaw Street Dothan, Alabama 36301

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO 10 CFR 50, APPENDIX B, CRITERION II CONSUMABLES (GREASE, OIL AND GASKET MATERIAL) TO BE USED IN SAFETY RELATED SYSTEMS ARE NOT FULLY IDENTIFIED OR CONTROLLED. SINCE NO IMPROPER USAGE WAS IDENTIFIED, A LOWER LEVEL OF SEVERITY IS ASSIGNED.
(8014 DEFICIENCY)

CONTRARY TO 10 CFR 50, APPENDIX B, CRITERION V AND THE LICENSEE'S PROCEDURE FNP-0-AP-12, THE CONTAINER THAT THE WELD RODS WERE STORED IN WAS NOT LABELED TO PREVENT THEIR USE IN CRITICAL SYSTEMS. SINCE NO IMPROPER USE OF WELD RODS WAS FOUND, A LOWER LEVEL OF SEVERITY IS ASSIGNED.
(8014 DEFICIENCY)

CONTRARY TO 10 CFR 50, APPENDIX B, CRITERION XVI INADEQUACIES WERE IDENTIFIED IN APRIL 1978 DURING A BI-WEEKLY QA AUDIT. FNP-NC-18-79/7(18), HOWEVER THE ESTIMATED COMPLETION DATE OF THESE INADEQUACIES WAS GIVEN AS 5/18/82. THE QA DEPARTMENT ACCEPTED THIS CORRECTIVE ACTION. THIS IS AN EXAMPLE OF LACK OF PROMPT CORRECTIVE ACTION.
(8014 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

REFUELING OUTAGE SCHEDULED TO BEGIN IN NOVEMBER, 1980.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

NORMAL OPERATIONS.

LAST IE SITE INSPECTION DATE: AUGUST 27-29, 1980

INSPECTION REPORT NO: 50-348/80-25

SUMMARY

INSPECTION SUMMARY

INSPECTION JUNE 9-13 AND 16-20 (80-14): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 83 INSPECTOR-HOURS ON SITE AND AT THE COMPANY OFFICES BY 4 REGION BASED INSPECTORS. OVERALL MANAGEMENT WAS REVIEWED BY INSPECTING 36 ASPECTS OF THE OVERALL QUALITY ASSURANCE PROGRAM WITHIN THE FOLLOWING 22 AREAS: RECEIPT, STORAGE AND HANDLING OF EQUIPMENT AND MATERIALS; QA/QC ADMINISTRATION; QA FOR STARTUP TESTING PROGRAM; QA PROGRAM ANNUAL REVIEW; MAINTENANCE; DESIGN CHANGES AND MODIFICATIONS; SURVEILLANCE TESTING; PROCUREMENT CONTROL; RECORDS; TEST AND EXPERIMENTS; TEST AND MEASURING EQUIPMENT; OFFSITE REVIEW; OPERATING STAFF TRAINING; TRAINING AND RETRAINING OF NON-LICENSED PERSONNEL; REQUALIFICATION TRAINING; AUDITS; DOCUMENT CONTROL; OPERATIONAL STAFFING; PREOPERATIONAL TEST RECORDS; OFFSITE SUPPORT STAFF; HOUSEKEEPING AND CLEANLINESS; AND, ORGANIZATION AND ADMINISTRATION. OF THE 22 AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN 20 AREAS; THREE ITEMS OF NONCOMPLIANCE WERE FOUND IN TWO AREAS (INFRACTION - FAILURE TO PROVIDE DEFINITIVE OR PROMPT CORRECTIVE ACTION FOR IDENTIFIED MATERIAL STORAGE INADEQUACIES; DEFICIENCY - FAILURE - TO FOLLOW PROCEDURE).

INSPECTION JULY 27-31 (80-21): THE INSPECTION INVOLVED 62 INSPECTOR-HOURS ON SITE BY TWO NRC INSPECTORS. THE INSPECTION WAS BEGUN DURING AN OFFSHIFT PERIOD; SIX INSPECTION HOURS WERE ACCOMPLISHED DURING OFFSHIFT PERIODS. AREAS INSPECTED INCLUDED REVIEW OF SECURITY PLAN; SECURITY PROGRAM AUDIT; LIGHTING; ALARM STATIONS; DETECTION AIDS; COMMUNICATIONS; COMPENSATORY MEASURES AND FOLLOWUP ON PREVIOUS INSPECTION FINDINGS (UNRESOLVED ITEM). OF THE EIGHT AREAS EXAMINED DURING THE INSPECTION, ONE APPARENT ITEM OF NONCOMPLIANCE WAS IDENTIFIED IN ONE AREA. SECURITY ORGANIZATION - SOME MEMBERS OF THE SECURITY FORCE WERE NOT PROPERLY QUALIFIED TO PERFORM SECURITY FUNCTIONS.

REPORTS RECEIVED FROM LICENSEE

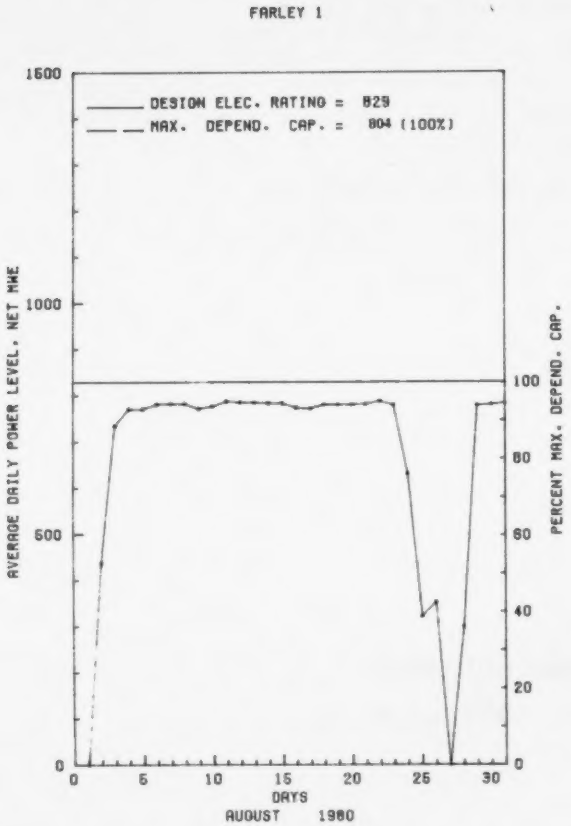
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-038/ 03L-0	07/12/80	08/08/80	PRESSURIZER PRESSURE LOW PRESSURE TRIP SETPOINT WAS FOUND LOW DURING SURVEILLANCE
80-039/ 03L-0	07/15/80	08/08/80	RIVER WATER TRAIN "A" INOPERABLE DUE TO PUMP NUMBER "8" BEING INOPERABLE
80-040/ 03L-0	07/17/80	08/15/80	DIESEL GENERATOR 2C WAS INOPERABLE
80-041/ 03L-0	07/08/80	08/07/80	CONTAINMENT SPRAY SURVEILLANCE TEST NOT PERFORMED
80-042/ 03L-0	08/01/80	08/15/80	REACTOR COOLANT PUMP BUS UNDERVOLTAGE RELAY TRIP VALVE INCORRECT
80-043/ 03L-0	07/17/80	08/15/80	DIESEL GENERATOR 1C WAS INOPERABLE

REVIEWED BY OMPA LOEB: T.C. CINTULA **DATE:** 10•6•80

REVIEWED BY NRR DL: E. A. Reeves **DATE:** 10-8-80

FARLEY 1

AVERAGE POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-348
2. Utility Contact: W. G. Hairston,III (205) 899-5156
3. Licensed Thermal Power (MWt): 2652
4. Nameplate Rating (Gross MWe): 1045 x 0.85 = 888
5. Design Electrical Rating (Net MWe): 829
6. Maximum Dependable Capacity (Gross MWe): 845
7. Maximum Dependable Capacity (Net MWe): 804
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

ITEM 21 CALCULATED WITH A WEIGHTED MDC

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	5,855.0	24,119.0
12. Hours Reactor Was Critical	701.5	4,707.6	15,643.1
13. Reactor Reserve Shutdown Hours	42.5	946.7	2,352.0
14. Hours Generator On-Line	676.2	4,606.3	15,196.8
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,694,628	11,677,110	38,275,125
17. Gross Electrical Energy (MWH)	526,020	3,722,960	12,246,998
18. Net Electrical Energy (MWH)	495,230	3,507,232	11,511,240
19. Unit Service Factor	90.9	78.7	63.0
20. Unit Availability Factor	90.9	78.7	63.0
*21. Unit Capacity Factor (MDC Net)	82.8	74.5	59.4
22. Unit Capacity Factor (DER Net)	80.3	72.3	57.6
23. Unit Forced Outage Rate	9.1	5.6	7.0

24. Shutdowns Scheduled Over Next 6 Months (Type,Date,and Duration of Each):
REFUELING OUTAGE - NOVEMBER, 1980, APPROXIMATELY 8 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
051	08/01/80	F	23.5	A	4		HA	PIPEXX	CONTINUATION OF UNIT OUTAGE FROM 07/31/80 TO REPAIR ELECTRO-HYDRAULIC FLUID LINE BREAK TO #4 GOVERNOR VALVE ON THE HIGH PRESSURE MAIN TURBINE.
052	08/02/80	F	.0	A	5		HD	INSTRU	UNIT HELD AT 750 MWE TO TROUBLESHOOT CONTROLLER FOR THE GLAND SEAL STEAM CONDENSER BYPASS VALVE.
053	03/09/80	S	.0	B	5		HA	VALVEX	UNIT REDUCED TO AND HELD AT 742 MWE FOR MAIN TURBINE GOVERNOR VALVE TESTING.
054	08/16/80	S	.0	B	5		HA	VALVEX	UNIT WAS REDUCED TO AND HELD AT 750 MWE FOR MAIN TURBINE GOVERNOR VALVE TESTING.
055	08/23/80	S	.0	B	5		HA	VALVEX	UNIT WAS REDUCED TO AND HELD AT 750 MWE FOR MAIN TURBINE GOVERNOR VALVE TESTING.
056	08/24/80	F	.0	A	5		HH	VALVEX	UNIT WAS REDUCED TO AND HELD AT LESS THAN 50% REACTOR POWER TO REPLACE THE LINEAR VOLTAGE DIFFERENTIAL TRANSFORMER ON THE LOW PRESSURE GOVERNOR VALVE FOR 1A SGFP.
057	08/27/80	F	12.1	G	3		EB	CKTBRK	UNIT TRIPPED WHEN BREAKER DH01 WAS RACKED OUT BY MISTAKE. THIS CAUSED BOTH SGFPS TO TRIP.
058	08/26/80	F	9.0	A	3		HH	VALVEX	UNIT TRIPPED ON LO-LO LEVEL IN B STEAM GENERATOR, WHILE ATTEMPTING TO PUT 1A SGFP ON-LINE AND REMOVE 1B SGFP FROM SERVICE TO TROUBLESHOOT AN OSCILLATING GOVERNOR VALVE.
059	08/27/80	F	23.2	A	1		HJ	XXXXXX	UNIT WAS MANUALLY SHUTDOWN TO REPLACE A CRACKED DISC ON #2 LOW PRESSURE TURBINE.

SUMMARY

FARLEY 1 OPERATED WITH FOUR OUTAGES AND FIVE REDUCTIONS LISTED IN DETAIL ABOVE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

FITZPATRICK

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	New York/Oswego 8 miles NE of Oswego
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2436
5.	Design Electrical Rating (MWE Net)	821
6.	Date of Initial Criticality	November 17, 1974
7.	Date of Commercial Operation	July 28, 1975
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Lake Ontario

Utility & Contractor Information

10.	Licensee	Power Authority of the State of New York
11.	Corporate Address	10 Columbus Circle New York, New York 10019
12.	Corporate Contact	J. P. Bayne, Manager of Nuclear Operations
13.	Architect/Engineer	Stone & Webster
14.	Nuclear Steam Supply System	GE
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Section Chief	H. B. KISTER
19.	IE Resident Inspector	W. H. BAUNACK (315) 342-4907 FTS: 264-3311
20.	Licensing Project Manager	P. Polk (X7872)
21.	Docket Number	50-333
22.	License Number/Date of Issue	DPR-59, 10/17/74
23.	Public Document Room	State University College of Oswego Penfield Library - Documents Oswego, NY 13126 (315)341-2323

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO TECHNICAL SPECIFICATION 2.3.B.2, REGARDING AIRBORNE EFFLUENTS ON MAY 12-13, 1980, THE RELEASE RATE FOR HALOGENS AND PARTICULATES WITH HALF-LIVES GREATER THAN EIGHT DAYS EXCEEDED THE LIMIT BY A FACTOR OF 1.3.
(8009 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS:

THE PLANT IS OPERATING AT ABOUT 75% POWER WITH AN OFFGAS RELEASE RATE OF 1630 MICROCURIES PER SECOND AT THE SJAE AND A STACK RELEASE RATE OF 835 MICROCURIES PER SECOND.

LAST IE SITE INSPECTION DATE: 8/28/80

INSPECTION REPORT NO: 50-333/80-12

SUMMARY

INSPECTION SUMMARY

50-333/80-06 - APR 14-18, 23-25: ROUTINE, UNANNOUNCED INSPECTION BY TWO REGION-BASED INSPECTORS (122 HRS) OF: SECURITY PLAN, SECURITY PROGRAM AUDIT, SECURITY ORGANIZATION, PHYSICAL BARRIERS, (PA), (VA), LIGHTING, ACCESS CONTROLS, ALARM SYSTEMS, DETECTION AIDS, ASSESSMENT AIDS, COMMUNICATIONS, TESTING AND MAINTENANCE, COMPENSATORY MEASURES, POWER SUPPLY, RESPONSE, LOCKS, KEYS AND COMBINATIONS, AND RECORDS AND REPORTS, FOLLOWUP ON PREVIOUS ITEMS OF NONCOMPLIANCE, IE BULLETINS NOS. 77-18 AND 79-16 AND IE CIRCULAR NO. 79-03/78-17. TWENTY ITEMS OF NONCOMPLIANCE WERE IDENTIFIED: (1) FAILURE TO FORWARD A CHANGE TO THE PHYSICAL SECURITY PLAN; (2) FAILURE TO SCREEN ALL INDIVIDUALS WHO HAD BEEN GRANTED UNESCORTED ACCESS TO THE PROTECTED AREA; (3) FAILURE TO PROPERLY TRAIN SECURITY FORCE PERSONNEL IN ACCORDANCE WITH THE PHYSICAL SECURITY PLAN; (4) FAILURE TO PROPERLY TRAIN CONTROL ROOM OPERATORS ON THE OPERATION OF THE SECONDARY ALARM AND TO DOCUMENT SUCH TRAINING; (5) FAILURE TO PROVIDE PROTECTION TO A PORTION OF THE PROTECTED AREA BARRIER THAT WAS ADJACENT TO THE SECURITY BUILDING; (6) FAILURE TO INSURE THAT THE EMERGENCY EXIT DOOR COULD ONLY BE OPENED FROM THE INSIDE; (7) FAILURE TO INSTALL STANDARD LOCKS AND ALARMS AND MAINTAIN THE DOUBLE BARRIER CONCEPT ON ONE VITAL AREA; (8) FAILURE TO HAVE AN INDIVIDUAL IN A BULLET RESISTANT STRUCTURE TO GRANT UNESCORTED ACCESS TO THE PROTECTED AREA; (9) FAILURE TO PURGE THE SECURITY PHOTO BADGES AFTER THEY HAD EXPIRED; (10) FAILURE TO CONDUCT "HANDS-ON" SEARCHES OF 5% SELECTED RANDOMLY); (11) FAILURE TO USE ARMED PERSONNEL FOR VEHICLE ESCORTS INTO THE PROTECTED AREA; (12) FAILURE TO INSURE THAT AN ALARM OCCURS EACH TIME AN UNAUTHORIZED KEY CARD IS USED; (13) FAILURE TO INSURE THAT THE CENTRAL ALARM STATION WAS NOT VISIBLE FROM OUTSIDE OF THE PROTECTED AREA; (14) FAILURE TO INSURE THAT THE RADIATION PATTERNS OF ADJACENT SENSOR ZONES OVERLAP; (15) FAILURE TO HAVE AN OPERABLE BACK UP COMMUNICATIONS SOURCE; (16) FAILURE TO MAINTAIN THE BULLET RESISTING PROPERTIES OF THE ACCESS CONTROL FUNCTION AT THE AUXILIARY ACCESS CONTROL POINT; (17) FAILURE TO MAINTAIN THE RADIATION DETECTORS IN OPERABLE CONDITION; (18) FAILURE TO MAINTAIN ADMINISTRATIVE CONTROLS ESTABLISHED FOR THE ISSUANCE OF SECURITY KEYS; (19) FAILURE TO ADEQUATELY MAINTAIN THE VISITOR REGISTRATION LOG; (20) FAILURE TO ADEQUATELY MAINTAIN VITAL AREA PATROL LOG.

50-333/80-09 - MAY 14-16: SPECIAL, UNANNOUNCED INSPECTION BY TWO REGION-BASED INSPECTORS (32 HRS) TO FOLLOWUP ON THE SITE RELEASE OF RADIOIODINE ABOVE THE TECHNICAL SPECIFICATION RELEASE RATE LIMIT. ONE ITEM OF NONCOMPLIANCE WAS IDENTIFIED: FAILURE TO ADHERE TO TECHNICAL SPECIFICATIONS REGARDING RELEASE RATES.

50-333/80-11 - JUN 23-27: ROUTINE, UNANNOUNCED INSPECTION BY ONE REGION-BASED INSPECTOR (41 HRS) OF INSERVICE INSPECTION ACTIVITIES, WALK THROUGH INSPECTION OF TORUS AND INSPECTION OF LICENSEE RESPONSE TO IE BULLETIN NO. 80-13. NO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

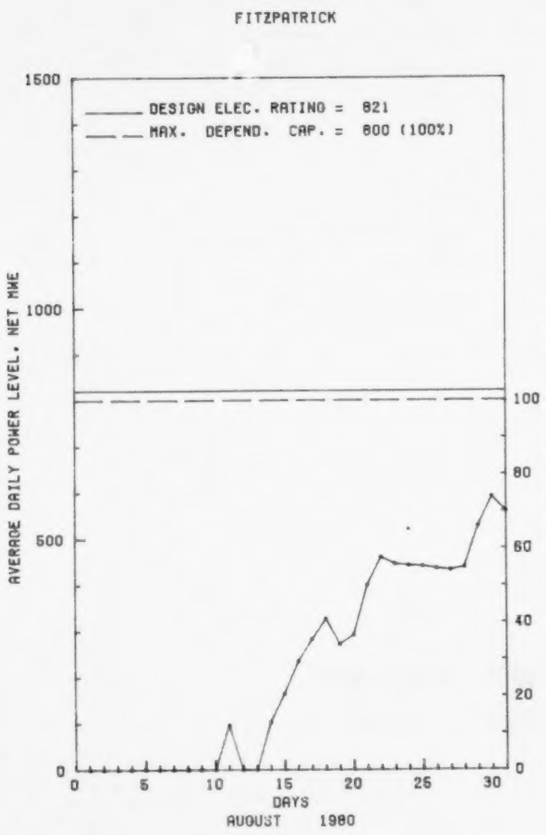
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-052/ 03L	05/28/80	06/25/80	REFUEL BRIDGE INTERLOCK FAILURE.
80-053/ 03L	05/29/80	06/25/80	LATE PORC REVIEW OF A TEMPORARY CHANGE TO A PROCEDURE.
80-054/ 04L	05/30/80	06/27/80	REACTOR BUILDING VENTILATION EXHAUST NOT MONITORED FOR 16 HOUR PERIOD.
80-055/ 03L	06/07/80	07/07/80	IRM INSTRUMENT DRIFT.
80-056/ 03L	06/10/80	07/10/80	EMERGENCY DIESEL GENERATOR TRIPPED DURING SURVEILLANCE TESTING.
80-057/ 03L	06/28/80	07/22/80	LOW SPECIFIC GRAVITY OF TWO DIESEL FIRE PUMP BATTERY CELLS.
80-058/ 03L	06/28/80	07/22/80	APRM SURVEILLANCE MISSED WHILE SHUTDOWN.
80-059/ 03L	07/03/80	07/22/80	MANY CONTROL ROD DRIVE HYDRAULIC CONTROL UNITS PRESSURE SWITCHES, LEVEL SWITCHES AND PRESSURE INDICATORS OUT OF CALIBRATION IDENTIFIED DURING SURVEILLANCE TESTING.
80-060/ 03L	07/10/80	07/30/80	TORUS LEVEL INSTRUMENT NOT INDICATING CORRECTLY IDENTIFIED DURING SURVEILLANCE TESTING.

REVIEWED BY OMPA LOEB: John R Crooks **DATE:** 10/6/80

REVIEWED BY NRR DL: Ken Eccleston **DATE:** 10/8/80

FITZPATRICK

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-333
2. Utility Contact: E. Zufelt (315) 342-3840
3. Licensed Thermal Power (MWt): 2436
4. Nameplate Rating (Gross MWe): 981 x 0.9 = 883
5. Design Electrical Rating (Net MWe): 821
6. Maximum Dependable Capacity (Gross MWe): 830
7. Maximum Dependable Capacity (Net MWe): 800
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

ITEM 21 CALCULATED WITH A WEIGHTED MDC

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	5,855.0	44,688.0
12. Hours Reactor Was Critical	499.0	3,435.1	36,035.3
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	435.0	3,328.0	29,066.0
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	532,260	6,394,224	57,582,418
17. Gross Electrical Energy (MWH)	171,470	2,190,490	19,544,190
18. Net Electrical Energy (MWH)	164,780	2,118,150	18,908,725
19. Unit Service Factor	58.5	56.8	65.0
20. Unit Availability Factor	58.5	56.8	65.0
*21. Unit Capacity Factor (MDC Net)	27.7	45.2	57.7
22. Unit Capacity Factor (DER Net)	27.0	44.1	51.5
23. Unit Forced Outage Rate	.0	4.8	19.4

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
SNUBBER INSPECTION & MAINTENANCE OUTAGE - 01/81 - 10 DAYS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
14	05/07/80	S	257.0	C	4		RC	FUELXX	REFUELING OUTAGE (CUMULATIVE HOURS 2309).
15	08/11/80	S	52.0	H	2				SCRAM TEST AND TURBINE TEST PER NRC BULLETIN 80-17.

SUMMARY

THE UNIT COMPLETED ITS REFUELING OUTAGE ON 08/11/80 AND NRC REQUIRED TESTING PER BULLETIN 80-17 ON 08/14/80. IT THEN PROCEEDED TO INCREASE POWER TO 70% AT THE MONTH'S END.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

FORT CALHOUN

FACILITY DATA

Facility Description

1. Location (state/county) (nearest population center)	Nebraska/Washington 19 miles N of Omaha
2. Electric Reliability Council	Mid-Continent Area Reliability Coordination Agreement
3. Type of Reactor	PWR
4. Power Level (MWT)	1500
5. Design Electrical Rating (MWE Net)	457
6. Date of Initial Criticality	August 6, 1973
7. Date of Commercial Operation	June 20, 1974
8. Condenser Cooling Method	Once Through
9. Condenser Cooling Water Source	Missouri River

Utility & Contractor Information

10. Licensee	W. C. Jones Division Manager – Production Operations Omaha Public Power District
11. Corporate Address	1623 Harney Street Omaha, Nebraska 68102
12. Corporate Contact	T. E. Short Assistant General Manager
13. Architect/Engineer	Gibbs & Hill, Inc.
14. Nuclear Steam Supply System	Combustion Engineering
15. Constructor	Gibbs & Hill, Durham & Richardson, Inc.

Regulatory Information

16. IE Region responsible	IV
17. IE Branch Chief	G. L. Madsen
18. IE Section Chief	D. M. Hunnicutt
19. IE Resident Inspector	D. Kelley
20. Licensing Project Manager	P. C. Wagner
21. Docket Number	50-285
22. License Number/Date of Issue	DPR-40, 5/24/73
23. Public Document Room	W. Dale Clark Library 215 S. 15th Street Omaha, Nebraska 68102

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

FACILITY ITEMS (PLANS AND PROCEDURES):

MANAGERIAL ITEMS:

PLANT STATUS:

No significant information

LAST IE SITE INSPECTION DATE: July 28 - 31, 1980

INSPECTION REPORT NO: 50-285/80-12

SUMMARY

INSPECTION SUMMARY

Inspection on June 30 - July 29, 1980 (80-11): Routine resident inspection of (1) Operational Safety Verification; (2) Post-refueling Tests; and (3) Observation of Annual Emergency Drill. The inspection of on-site activities by the resident inspector included 176 inspector hours. In the three (3) areas examined, no deviations or items of noncompliance were identified.

Inspection on July 28-31, 1980 (80-12): Routine, unannounced inspection of the licensee's: (1) organizational structure and the qualifications/experience of personnel assigned to key management postions: and (2) the calibration program for safety-related instrument systems. The inspection consisted of forty (40) hours on-site and one (1) hour at the district office by two (2) inspectors. Of the two (2) areas inspected, no items of noncompliance or deviations were noted. No new enforcement actions were identified by the licensee performance evaluation. As a result of the SALP evaluation no increase in the scope of frequency of inspection effort is planned.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

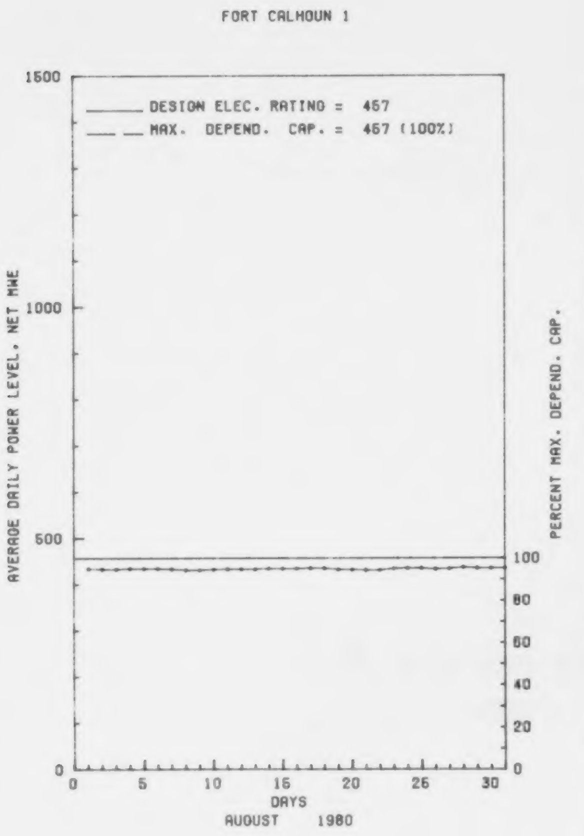
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-12 03L-0	7/8/80	7/11/80	Reactor coolant flow trip setpoints had drifted out of specification
80-13 03L-0	7/7/80	7/8/80	During normal power opeation, channel A DC sequencer SI-1 had a de-energized "on-auto" light associated with safeguards equipment from buss 1B3B
80-14 03L-0	7/9/80	8/1/80	Emission heater and alarm circuit for transformer on Diesel Generator 1 failed
80-15 03L-0	7/10/80	7/18/80	The plus or minus 15 volt power supply for trip unit 4 (Steam Generator Level) failed
80-16 03L-0	6/21/80	7/21/80	A weld cracked on the suction side of charging pump CH-1B

REVIEWED BY OMPA LOEB: MR Beebe **DATE:** 10/06/80

REVIEWED BY NRR DL: P C Wagner **DATE:** 10/07/80

FORT CALHOUN

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

ITEM 21 CALCULATED WITH A WEIGHTED MDC.

1. Docket Number: 50-285
2. Utility Contact: B. J. Hickle (402) 536-4413
3. Licensed Thermal Power (MWt): 1420
4. Nameplate Rating (Gross MWe): 591 x 0.85 = 502
5. Design Electrical Rating (Net MWe): 457
6. Maximum Dependable Capacity (Gross MWe): 481
7. Maximum Dependable Capacity (Net MWe): 457
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	5,855.0	60,792.0
12. Hours Reactor Was Critical	744.0	2,449.1	46,850.7
13. Reactor Reserve Shutdown Hours	.0	.0	1,309.5
14. Hours Generator On-Line	744.0	2,387.4	45,838.5
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,049,153	3,166,746	56,401,773
17. Gross Electrical Energy (MWH)	339,426	1,027,932	18,688,162
18. Net Electrical Energy (MWH)	322,571	974,198	17,601,715
19. Unit Service Factor	100.0	40.8	75.4
20. Unit Availability Factor	100.0	40.8	75.5
*21. Unit Capacity Factor (MDC Net)	94.9	36.4	63.6
22. Unit Capacity Factor (DER Net)	94.9	36.4	63.6
23. Unit Forced Outage Rate	.0	4	4.3

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	None								

SUMMARY

FORT CALHOUN OPERATED NEAR FULL POWER FOR THE ENTIRE MONTH OF AUGUST WITH NO OUTAGES OR REDUCTIONS.
POWER LEVEL INCREASE TO 1500 MWT WAS AUTHORIZED ON AUGUST 15, 1980.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

NRC POWER RESTRICTIONS: Restricted to 70% of Licensed Thermal Power.

FORT ST. VRAIN

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Colorado/Weld 35 miles N of Denver (near Platteville)
2.	Electric Reliability Council	Western Systems Coordinating Council
3.	Type of Reactor	HTGR
4.	Power Level (MWT)	842
5.	Design Electrical Rating (MWE Net)	330
6.	Date of Initial Criticality	January 31, 1974
7.	Date of Commercial Operation	July 1, 1979
8.	Condenser Cooling Method	Towers
9.	Condenser Cooling Water Source	South Platte River/Shallow Wells

Utility & Contractor Information

10.	Licensee	Public Service Company of Colorado
11.	Corporate Address	P. O. Box 840 Denver, Colorado 80201
12.	Corporate Contact	C. K. Millen Senior Vice President
13.	Architect/Engineer	Sargent & Lundy
14.	Nuclear Steam Supply System	General Atomic Company
15.	Constructor	General Atomic Company

Regulatory Information

16.	IE Region responsible	IV
17.	IE Branch Chief	G. L. Madsen
18.	IE Section Chief	T. F. Westerman
19.	IE Resident Inspector	M. W. Dickerson (303) 785-2281
20.	Licensing Project Manager	G. Kuzmycz
21.	Docket Number	50-267
22.	License Number/Date of Issue	DPR-34, 12/21/73
23.	Public Document Room	Greeley Public Library City Complex Building Greeley, Colorado 80631

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO THE REQUIREMENTS OF TS 7.4.D AND ADMINISTRATIVE PROCEDURE ADM-07, THREE INDIVIDUALS WERE OBSERVED IN AN RWP AREA WITHOUT THE FULL PROTECTIVE CLOTHING REQUIRED BY THE RWP (NO. 2365).
(8015 INFRACTION)

10 CFR APPEN B, CRIT V REQUIRES THAT ACTIVITIES AFFECTING QUALITY BE PRESCRIBED AND ACCOMPLISHED IN ACCORDANCE WITH INSTRUCTIONS. FSV FSAR, APPEN B, SECT B.5.6, REQUIRES THAT PROCEDURES PROVIDE FOR CONTROL AND ACCOUNTABILITY ON DOCUMENTS USED TO CONTROL NUCLEAR RELATED ACTIVITIES AND THAT THESE DOCUMENTS BE KEPT UP-TO-DATE. FSV-ENG-1, STEP 5.7 REQUIRES THAT DESIGN DOCUMENTS BE IDENTIFIED WHEN THEY ARE BEING AFFECTED BY AN UNINCORPORATED CHANGE NOTICE. CONTRARY TO THE ABOVE, THE DESIGN DOCUMENTS LISTED IN CHANGE NOTICES 1156 AND 1202 WERE NOT IDENTIFIED AS BEING AFFECTED BY AN UNINCORPORATED CHANGE NOTICE.
(8016 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

Areas related to potential equipment modification, which are under review and discussion with the NRC include: helium circulator auxiliaries; accident analysis relating to outlet temperature dispersion and orifice settings; hangers for 2" and smaller piping; and plant oscillations.

FACILITY ITEMS (PLANS AND PROCEDURES):

MANAGERIAL ITEMS:

PLANT STATUS:

Plant shut down for completion of surveillance and maintenance activities

LAST IE SITE INSPECTION DATE: July 1 - July 31, 1980

INSPECTION REPORT NO: 50-267/80-15

SUMMARY

INSPECTION SUMMARY

Inspection July 1 - 31, 1980 (80-15): Routine, announced inspection of plant operations; review of event reports; follow-up on inspector identified and unresolved items; surveillance; maintenance; operational safety verification; and review of bulletins/circulars. Within the seven (7) areas inspected, one item of noncompliance was identified (infraction - failure to comply with requirements of RWP.

REPORTS RECEIVED FROM LICENSEE

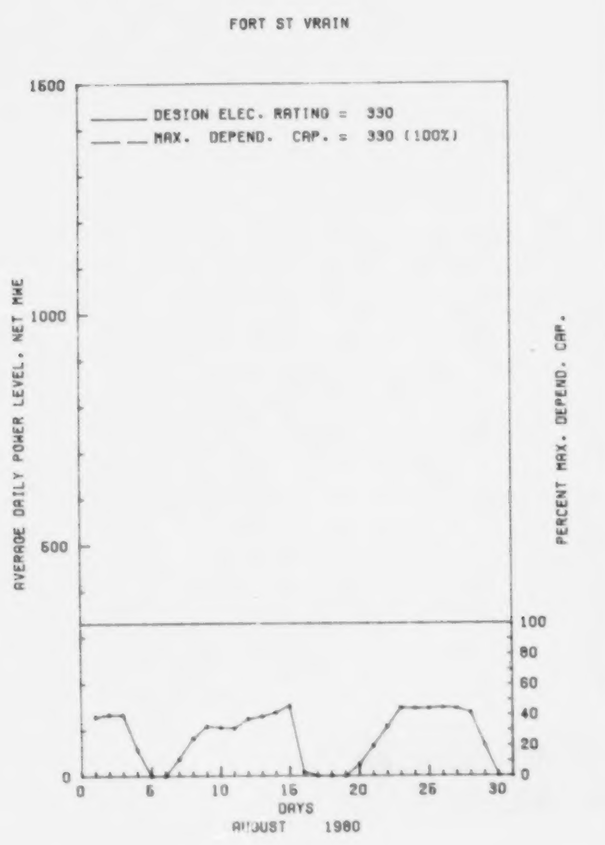
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-36 03L-0	6/25/80	7/25/80	Total oxidants greater than 10 ppm and dewpoint greater than limits at power (LCO 4.2.10, 4.2.11)
80-37 03L-0	7/11/80	8/8/80	Only one PCRV liner cooling loop operating (LCO 4.2.13)
80-38 03L-0	7/9/80	8/8/80	Waste water discharged to plant liquid effluent system. (LCO 1.1)
80-39 03L-0	7/21/80	8/20/80	Liquid nitrogen storage tank level less than 650 gal (LCO 4.2.12)
80-40 03L-0	7/12/80	8/11/80	Sample required by SR NR 1.1 not taken
80-41 03L-0	7/24/80	8/22/80	B/MD switches out of tolerance (1 of 12) (LCO 4.4.1)
80-42 03L-0	7/25/80	8/22/80	Two inoperable reserve shutdown units (LCO 4.1.1)
80-43 03L-0	7/22/80	8/27/80	Primary oxidants greater than 10 ppm, core avg. outlet temperature greater than 1200 deg f (LCO 4.2.10)

REVIEWED BY OMPA LOEB: T.C. Cintula DATE: 10·6·80

REVIEWED BY NRR DL: ______________ DATE: ______________

FORT ST. VRAIN

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-267
2. Utility Contact: J. W. GAHM (303) 785-2253
3. Licensed Thermal Power (MWt): 842
4. Nameplate Rating (Gross MWe): 403 x 0.85 = 343
5. Design Electrical Rating (Net MWe): 330
6. Maximum Dependable Capacity (Gross MWe): 342
7. Maximum Dependable Capacity (Net MWe): 330
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): 231
10. Reasons for Restrictions, If Any:
NRC RESTRIC. 70% PENDING RESOL OF TEMP FLUC

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	5,855.0	10,272.0
12. Hours Reactor Was Critical	619.4	4,343.1	6,875.9
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	517.7	2,974.9	3,957.1
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	217,522	1,350,115	1,828,070
17. Gross Electrical Energy (MWH)	66,793	429,129	569,925
18. Net Electrical Energy (MWH)	60,272	393,379	516,963
19. Unit Service Factor	69.6	50.8	38.5
20. Unit Availability Factor	69.6	50.8	38.5
21. Unit Capacity Factor (MDC Net)	24.5	20.4	15.3
22. Unit Capacity Factor (DER Net)	24.5	20.4	15.3
23. Unit Forced Outage Rate	24.5	30.1	44.0

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 10/01/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-15	08/04/80	F	65.2	A	3				REACTOR SCRAM AND TURBINE TRIP OCCURRED DUE TO LOSS OF ALL FOUR CIRCULATORS.
80-16	08/16/80	F	102.7	A	2	80-45	AD	PIPEXX	TURBINE TRIPPED AND REACTOR MANUALLY SCRAMMED AS A CONSERVATIVE MEASURE DUE TO RUPTURE OF A HYDRAULIC OIL SUPPLY LINE.
80-17	08/29/80	S	58.4	B	1				FOLLOWING A TURBINE RUNBACK AS A RESULT OF PERSONNEL WORKING ON THE EHC SYSTEM, THE TURBINE WAS MANUALLY TRIPPED AND THE REACTOR SHUTDOWN. THIS BEGAN THE SCHEDULED SHUTDOWN FOR SURVEILLANCE TESTING.

SUMMARY

THE UNIT OPERATED AT 50% POWER DURING THE MONTH AND INCURRED THREE OUTAGES - TWO FOR TURBINE TRIPS AND ONE FOR SURVEILLANCE TESTING. THE UNIT CONCLUDED THE MONTH IN A MAINTENANCE SHUTDOWN.

LEGEND

[1] Type
F-Forced
S-Scheduled

[2] Reason:
A-Equipment Failure
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error
H-Other

[3] Method
1-Manual
2-Manual Scram
3-Automatic Scram
4-Continuations
5-Load Reduction
9-Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

GINNA

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	New York/Wayne 15 miles NE of Rochester
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	1520
5.	Design Electrical Rating (MWE Net)	470
6.	Date of Initial Criticality	November 8, 1969
7.	Date of Commercial Operation	July 1970
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Lake Ontario

Utility & Contractor Information

10.	Licensee	Rochester Gas & Electric Corporation
11.	Corporate Address	89 East Avenue Rochester, New York
12.	Corporate Contact	Leon D. White, Jr., Vice President Electric and Steam Production
13.	Architect/Engineer	Gilbert
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Section Chief	H. B. KISTER
19.	IE Resident Inspector	D. JOHNSON
20.	Licensing Project Manager	S. J. Nowicki
21.	Docket Number	50-244
22.	License Number/Date of Issue	DPR-18, 9/19/69
23.	Public Document Room	Rochester Public Library Business and Social Science Division 115 South Avenue Rochester, New York 14604

INSPECTION STATUS

ENFORCEMENT STATUS

TECHNICAL SPECIFICATION 6.11, "RADIATION PROTECTION PROGRAM", STATES, IN PART, "RADIATION CONTROL PROCEDURES SHALL BE PREPARED AND MADE AVAILABLE TO ALL STATION PERSONNEL OR OTHER PERSONS WHO MAY BE SUBJECT TO RADIATION EXPOSURE AT THE STATION...THE RADIATION PROTECTION PROGRAM SHALL BE ORGANIZED AND MAINTAINED TO MEET THE REQUIREMENTS OF 10 CFR PART 20....THE PROGRAM SHALL BE ADHERED TO FOR ALL OPERATIONS INVOLVING PERSONNEL RADIATION EXPOSURE."1. PROCEDURE HP-4.3, "WORK PERMIT USE", REVISION 14, REQUIRES IN SECTION 6.1.5 THAT EACH PERSON WORKING UNDER A RADIATION WORK PERMIT (RWP) SHALL COMPLY WITH THE RWP IN ALL RESPECTS. CONTRARY TO THE ABOVE, ON MAY 1, 1980, TWO INDIVIDUALS WORKING UNDER RWP NO. 1, AT MOTOR CONTROL PANEL 1-C, DID NOT COMPLY WITH THE RWP IN ALL RESPECTS, IN THAT THEY REMOVED THEIR PROTECTIVE WORK GLOVES, WHICH WERE REQUIRED BY THE RWP. 2. PROCEDURE A.1, "RADIATION CONTROL MANUAL", REVISION 14, REQUIRES IN SECTION VI.A., "ACCESS TO CONTROLLED AREAS", THAT SMOKING, CHEWING, OR EATING ARE STRICTLY PROHIBITED IN CONTROLLED AREAS. CONTRARY TO THE ABOVE, ON MAY 1, 1980, FOUR INDIVIDUALS WERE CHEWING GUM WHILE IN CONTROLLED AREAS (I.E., THE AUXILIARY BUILDING AND THE REACTOR CONTAINMENT).
(8006 INFRACTION)

CONTRARY TO GENERALLY ACCEPTABLE PRACTICE IN INDUSTRY, INDIVIDUALS WERE OBSERVED INTENTIONALLY HANDING CONTAMINATED AND POTENTIALLY CONTAMINATED MATERIALS WITHOUT WEARING PROTECTIVE GLOVES.
(8006 DEVIATION)

TECHNICAL SPECIFICATION 6.11, "RADIATION PROTECTION PROGRAM", STATES, IN PART, "RADIATION CONTROL PROCEDURES SHALL BE PREPARED AND MADE AVAILABLE TO ALL STATION PERSONNEL OR OTHER PERSONS WHO MAY BE SUBJECT TO RADIATION EXPOSURE AT THE STATION...THE RADIATION PROTECTION PROGRAM SHALL BE ORGANIZED AND MAINTAINED TO MEET THE REQUIREMENTS OF 10 CFR PART 20....THE PROGRAM SHALL BE ADHERED TO FOR ALL OPERATIONS INVOLVING PERSONNEL RADIATION EXPOSURE."1. PROCEDURE HP-4.3, "WORK PERMIT USE", REVISION 14, REQUIRES IN SECTION 6.1.5 THAT EACH PERSON WORKING UNDER A RADIATION WORK PERMIT (RWP) SHALL COMPLY WITH THE RWP IN ALL RESPECTS. CONTRARY TO THE ABOVE, ON MAY 1, 1980, TWO INDIVIDUALS WORKING UNDER RWP NO. 1, AT MOTOR CONTROL PANEL 1-C, DID NOT COMPLY WITH THE RWP IN ALL RESPECTS, IN THAT THEY REMOVED THEIR PROTECTIVE WORK GLOVES, WHICH WERE REQUIRED BY THE RWP. 2. PROCEDURE A.1, "RADIATION CONTROL MANUAL", REVISION 14, REQUIRES IN SECTION VI.A., "ACCESS TO CONTROLLED AREAS", THAT SMOKING, CHEWING, OR EATING ARE STRICTLY PROHIBITED IN CONTROLLED AREAS. CONTRARY TO THE ABOVE, ON MAY 1, 1980, FOUR INDIVIDUALS WERE CHEWING GUM WHILE IN CONTROLLED AREAS (I.E., THE AUXILIARY BUILDING AND THE REACTOR CONTAINMENT).
(8006 INFRACTION)

CONTRARY TO GENERALLY ACCEPTABLE PRACTICE IN INDUSTRY, INDIVIDUALS WERE OBSERVED INTENTIONALLY HANDING CONTAMINATED AND POTENTIALLY CONTAMINATED MATERIALS WITHOUT WEARING PROTECTIVE GLOVES.
(8006 DEVIATION)

10 CFR 50, APP. B, CRITERION VI STATES IN PART, "MEASURES SHALL BE ESTABLISHED TO CONTROL...DRAWINGS, INCLUDING CHANGES... THESE MEASURES SHALL INSURE CHANCES ARE DISTRIBUTED TO THE LOCATION WHERE THE ACTIVITY IS PERFORMED... CONTRARY TO THE ABOVE "AS-BUILT" DRAWING 33013-179 WAS RECEIVED IN THE CONTROL ROOM, 13 DAYS AFTER THE MODIFICATION WAS COMPLETED.
(8303 DEFICIENCY)

TECH SPEC 6.8.1 STATES IN PART, "WRITTEN PROCEDURES SHALL BE ESTABLISHED, IMPLEMENTED....COVERING THE ACTIVITIES REFERENCED BELOW: (A) THE APPLICABLE PROCEDURES RECOMMENDED IN APP. A OF R.G. 133..." ADMIN. PROCEDURE A-20 STATES IN PART, "TO VOID AN INCORRECT LOG ENTRY, DRAW ONE LINE THROUGH THE ENTRY AND THEN INITIAL THE CORRECTION..." CONTRARY TO THE ABOVE WRITE-OVERS WERE NOTED IN THE OFFICIAL RECORD FOR MAY 19, 20 21 AND 23, 1980.
(8008 INFRACTION)

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-006/ 01T	07/14/80	07/24/80	ENTERED TECH SPEC 3.2.2 LIMITING CONDITION FOR OPERATION POWER REDUCTION COMMENCED DUE TO BAST BORON CONCENTRATION OUT OF SPECIFICATION.

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS:

Operating at 100% power.

LAST IE SITE INSPECTION DATE: 8/1-31/80

INSPECTION REPORT NO: 50-244/80-11

SUMMARY

INSPECTION SUMMARY

50-244/80-08 - JUN 2-5: ROUTINE, UNANNOUNCED INSPECTION ON SITE AND AT THE CORPORATE OFFICES BY THREE REGION-BASED INSPECTORS AND ONE RESIDENT INSPECTOR (112 HRS) OF THE QUALITY ASSURANCE PROGRAM IMPLEMENTATION INCLUDING: QAP ANNUAL REVIEWS; DESIGN CHANGES AND MODIFICATIONS; DESIGN CHANGES AND MODIFICATIONS CONTROL PROGRAM; AUDIT PROGRAM IMPLEMENTATION; PREVIOUSLY IDENTIFIED ITEMS; AND, A PLANT/WAREHOUSE TOUR. TWO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED: FAILURE TO PROVIDE TIMELY "AS-BUILT" INFORMATION TO THE CONTROL ROOM; FAILURE TO FOLLOW WRITTEN PROCEDURE IN REGARD TO MAKING CORRECTIONS ON CONTROL ROOM LOGS.

50-244/80-09 - JUN 9 - JUL 31: ROUTINE INSPECTIONS BY THE RESIDENT INSPECTOR (52 HRS) OF PLANT OPERATIONS, INCLUDING TOURS OF THE FACILITY; LOG AND RECORD REVIEW; REVIEW OF LICENSEE EVENTS; OPERATIONAL SAFETY VERIFICATION; OBSERVATION OF PHYSICAL SECURITY; AND, QUALIFICATION OF THE TRAINING STAFF. NO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED

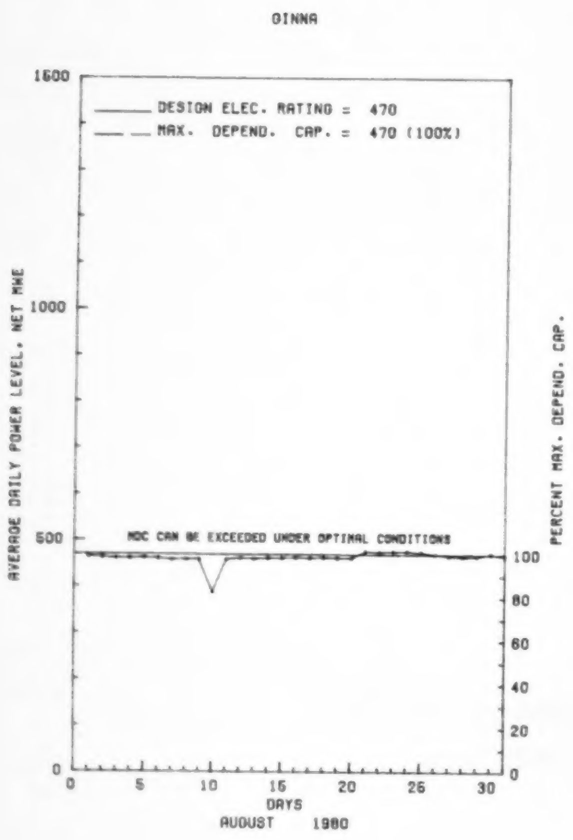
50-244/80-10 - JUL 6-11: ROUTINE, UNANNOUNCED PHYSICAL PROTECTION INSPECTION BY ONE REGION-BASED INSPECTOR (48 HRS) WHICH INCLUDED: SECURITY PLAN; SECURITY PROGRAM AUDIT; SECURITY ORGANIZATION; PHYSICAL BARRIERS (PROTECTED AREA); PHYSICAL BARRIERS (VITAL AREA); LIGHTING; ACCESS CONTROL (IDENTIFICATION, AUTHORIZATION AND BADGING); ACCESS CONTROL (SEARCH); ACCESS CONTROL (ESCORTING); ACCESS CONTROL (VITAL AREAS); ALARM STATIONS; DETECTION AIDS; ASSESSMENT AIDS; COMMUNICATIONS; TESTING AND MAINTENANCE; COMPENSATORY MEASURES; POWER SUPPLY; RESPONSE; LOCKS, KEYS AND COMBINATIONS; RECORDS AND REPORTS. ONE ITEM OF NONCOMPLIANCE WAS IDENTIFIED: FAILURE OF ADMITTANCE CONTROL HARDWARE TO ANNUNCIATE IN THE REQUIRED LOCATION.

REVIEWED BY OMPA LOEB: T.C. Cintula **DATE:** 10·6·80

REVIEWED BY NRR DL: S.J. Nowicki **DATE:** 10/8/80

GINNA

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-244
2. Utility Contact: A.E. MCNAMARA (315) 524-4446
3. Licensed Thermal Power (MWt): 1520
4. Nameplate Rating (Gross MWe): 608 x 0.85 = 517
5. Design Electrical Rating (Net MWe): 470
6. Maximum Dependable Capacity (Gross MWe): 490
7. Maximum Dependable Capacity (Net MWe): 470

CAPACITY FACTORS CALCULATED WITH A WEIGHTED AVERAGE

8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	5,855.0	94,367.0
12. Hours Reactor Was Critical	744.0	4,554.0	72,233.8
13. Reactor Reserve Shutdown Hours	.0	.0	1,621.8
14. Hours Generator On-Line	744.0	4,515.3	70,472.1
15. Unit Reserve Shutdown Hours	.0	.0	8.5
16. Gross Thermal Energy (MWH)	1,119,528	6,682,008	94,957,810
17. Gross Electrical Energy (MWH)	361,976	2,190,013	30,875,804
18. Net Electrical Energy (MWH)	343,959	2,081,928	29,243,476
19. Unit Service Factor	100.0	77.1	74.7
20. Unit Availability Factor	100.0	77.1	74.7
21. Unit Capacity Factor (MDC Net)	98.4	75.7	68.3
22. Unit Capacity Factor (DER Net)	98.4	75.7	68.3
23. Unit Forced Outage Rate	.0	.0	9.3

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
STEAM GEN. INSPECTION & MODIFICATION - 10/25 THRU 11/26/80.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	08/10/80	S	.0	B	5				TURBINE PROTECTOR AND CONTROL TESTS
	08/22/80	S	.0	B	5				AUXILIARY FEEDWATER P.T.
	08/26/80	F	.0	A	5				TURBINE RUNBACK.
	08/27/80	S	.0	B	5				AUXILIARY FEEDWATER P.T.

SUMMARY

THE UNIT OPERATED AT FULL POWER FOR 97% OF THE MONTH, WITH FOUR REDUCTIONS FOR TESTING AND TURBINE RUNBACK ON 08/10, 08/22, 08/26 AND 08/27/80.

LEGEND

[1] Type:
F–Forced
S–Scheduled

[2] Reason:
A–Equipment Failure
B–Maintenance or Test
C–Refueling
D–Regulatory Restriction
E–Operator Training & License Examination
F–Administrative
G–Operational Error
H–Other

[3] Method:
1–Manual
2–Manual Scram
3–Automatic Scram
4–Continuations
5–Load Reduction
9–Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

HADDAM NECK

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Connecticut/Middlesex 13 miles E of Meriden
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	1825
5.	Design Electrical Rating (MWE Net)	580
6.	Date of Initial Criticality	July 24, 1967
7.	Date of Commercial Operation	January 1, 1968
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Connecticut River

Utility & Contractor Information

10.	Licensee	Connecticut Yankee Atomic Power Co.
11.	Corporate Address	P. O. Box 270 Hartford, Connecticut
12.	Corporate Contact	D. Switzer, President
13.	Architect/Engineer	Stone & Webster
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Section Chief	R. Keimig
19.	IE Resident Inspector	T. H. Smith
20.	Licensing Project Manager	R. Caruso
21.	Docket Number	50-213
22.	License Number/Date of Issue	DPR-61, 12/27/74
23.	Public Document Room	Russell Library 119 Broad Street Middletown, Connecticut 06457

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO TECHNICAL SPECIFICATION 6.8.1: ANSI N18.7, PARAGRAPH 5.1.6 AND 5.3.5(3); PROC. QA 1.2-11.3, PARAGRAPH 4.5; PROC. QA 1.2-10.1, PARAGRAPH 5.1; PROC. QA 1.2-8.2, PARAGRAPHS 7.1 AND 7.2; PROC.QA 1.2-5.1, PARAGRAPHS 6.2.13 AND 6.2.23 - SAFETY RELATED MAINTENANCE DONE WITHOUT USE OF PROCEDURES AND NO POST MAINTENANCE INSTALLATION INSPECTIONS AND FUNCTIONAL TESTS; QA MATERIAL SERIAL NO. NOT RECORDED ON A WORK PERMIT; QA AUTHORIZATION AND WORK PERMITS NOT RECEIVING ADEQUATE FINAL QA REVIEW.
(8008 INFRACTION)

CONTRARY TO TECHNICAL SPECIFICATION 6.8.1; REGULATORY GUIDE 1.33, APPENDIX A, PARAGRAPH E; ANSI N1 8.7-1972, PARAGRAPH 5.33 - ALARM PROCEDURES WERE NOT PROVIDED; ALARM PROCEDURES DID NOT REFLECT THE CHANGES.
(8008 INFRACTION)

CONTRARY TO TECHNICAL SPECIFICATIONS 6.8.1 AND 6.10.1, AND PROCEDURE QA 1.2-5.1 RECORDS WERE NOT MAINTAINED FOR FOUR MAINTENANCE ACTIVITIES - 2 ELECTRICAL PM'S; REPAIR OF REACTOR COOLANT PUMP SEAL; AND SET PRESSURE DATA FOR TEST OF SAFETY VALVE PERFORMED IN CONJUNCTION WITH SAFETY VALVE MAINTENANCE.
(8008 DEFICIENCY)

CONTRARY TO TECHNICAL SPECIFICATION 6.5.1; 10 CFR 50, APPENDIX B, CRITERIA VI; ANSI N18.7-1972, 4.2.5; PROCEDURE ADM 1.1-1 - THE MINUTES OF PORC MEETINGS WERE INAPPROPRIATELY MAINTAINED; PORC MEETING CONDUCTED WITHOUT QUORUM; CHANGES TO TECHNICAL SPECIFICATIONS AND PROCEDURES APPROVED IN PORC MEETING CONDUCTED WITHOUT A QUORUM, AND IMPLEMENTED.
(8008 DEFICIENCY)

CONTRARY TO TECHNICAL SPECIFICATION 6.8.1; ANSI N18.7-1972, PARAGRAPH 6.2; QA 1.2-6.1, PARAGRAPH 7.1 - FIRE PROTECTION SYSTEMS TESTING PERFORMED WITH SUPERCEDED PROCEDURE; CHANGES NOT POSTED TO SEVERAL PROCEDURES; AND SUPERCEDED PROCEDURE NOT REMOVED FROM CONTROLLED COPY FILES.
(8008 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS:

PLANT OPERATING AT 100% POWER.

LAST IE SITE INSPECTION DATE: 8/25-29/80

INSPECTION REPORT NO: 50-213/80-15

SUMMARY

INSPECTION SUMMARY

NONE.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

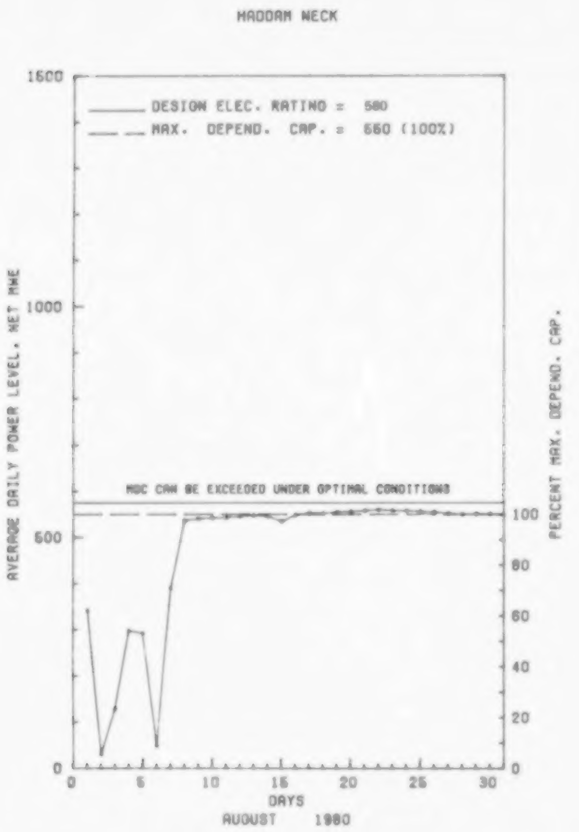
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE			

REVIEWED BY OMPA LOEB: T.C. Cintula **DATE:** 10·6·80

REVIEWED BY NRR DL: Ralph Caruso **DATE:** 10/8/80

HADDAM NECK

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

CAPACITY FACTORS CALCULATED WITH A WEIGHTED AVERAGE

1. Docket Number: 50-213
2. Utility Contact: DON ANDERSON (203) 267-2556 X274
3. Licensed Thermal Power (MWt): 1825
4. Nameplate Rating (Gross MWe): 667 x 0.9 = 600
5. Design Electrical Rating (Net MWe): 582
6. Maximum Dependable Capacity (Gross MWe): 577
7. Maximum Dependable Capacity (Net MWe): 555
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: ITEM 5-INSTALLED AN IMPROVED TURBINE ROTOR-1976
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	5,855.0	111,047.0
12. Hours Reactor Was Critical	727.5	3,858.9	95,055.6
13. Reactor Reserve Shutdown Hours	15.6	27.0	1,179.6
14. Hours Generator On-Line	710.7	3,767.6	90,787.8
15. Unit Reserve Shutdown Hours	0	0	369.9
16. Gross Thermal Energy (MWH)	1,185,536	6,382,816	156,523,955
17. Gross Electrical Energy (MWH)	371,992	2,096,615	51,451,879
18. Net Electrical Energy (MWH)	352,976	1,987,720	48,942,483
19. Unit Service Factor	95.5	64.3	81.7
20. Unit Availability Factor	95.5	64.3	82.0
*21. Unit Capacity Factor (MDC Net)	86.3	61.7	81.5
22. Unit Capacity Factor (DER Net)	81.8	58.5	75.2
23. Unit Forced Outage Rate	2.5	.8	7.8

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): TURBINE BALANCE AND MAINT. - 9/26/80 - APPROX. 3 DAYS
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-4	08/02/80	F	18.5	G	3				REACTOR AND TURBINE TRIP DUE TO TURBINE OVERSPEED. OVERSPEED TRIP SETTING IMPROPERLY ADJUSTED.
80-5	08/05/80	S	14.8	F	1				SHUTDOWN IN ORDER TO TIE IN LOOP 2 AFTER RCP #2 REPAIR.

SUMMARY

CONNECTICUT YANKEE HADDAM NECK OPERATED AT NEAR FULL POWER, WITH THE EXCEPTION OF TWO OUTAGES FOR OPERATIONAL ERROR AND MAINTENANCE.

LEGEND

[1] Type:
F-Forced
S-Scheduled

[2] Reason:
A-Equipment Failure
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error
H-Other

[3] Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Continuations
5-Load Reduction
9-Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source

NRC POWER RESTRICTIONS: NONE

HATCH 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Georgia/Appling 11 miles N of Baxley
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2436
5.	Design Electrical Rating (MWE Net)	777
6.	Date of Initial Criticality	September 12, 1974
7.	Date of Commercial Operation	December 31, 1975
8.	Condenser Cooling Method	Once Through With Forced Draft Cooling Towers
9.	Condenser Cooling Water Source	Altamaha River

Utility & Contractor Information

10.	Licensee	Georgia Power Company
11.	Corporate Address	270 Peachtree Street, NE Atlanta, Georgia 30303
12.	Corporate Contact	J. H. Miller Executive Vice President
13.	Architect/Engineer	Southern Services, Inc.
14.	Nuclear Steam Supply System	GE
15.	Constructor	Georgia Power Company

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. Lewis
18.	IE Section Chief	H. C. Dance
19.	IE Resident Inspector	R. F. Rogers (912) 367-9881
20.	Licensing Project Manager	D. Verrelli
21.	Docket Number	50-321
22.	License Number/Date of Issue	DPR-57, 8/6/74
23.	Public Document Room	Appling County Public Library Parker Street Baxley, Georgia 31513

INSPECTION STATUS

ENFORCEMENT STATUS

TECHNICAL SPECIFICATION 6.8.1 REQUIRE THAT PLANT OPERATING PROCEDURES BE IMPLEMENTED. THE HATCH RHR SERVICE WATER PROCEDURE REQUIRES THAT COOLING WATER VALVES TO RHRSW PUMP MOTORS BE OPEN. CONTRARY TO THIS, THE VALVES FOR A AND B RHRSW PUMPS WERE FOUND SHUT ON MAY 10, 1980. (8024 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

PLANT IS OPERATING AT FULL POWER.

LAST IE SITE INSPECTION DATE: AUGUST 13-14, 1980

INSPECTION REPORT NO: 50-321/80-34

SUMMARY

INSPECTION SUMMARY

INSPECTION JULY 7-11, 14-18, 22-24 (80-28): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 143 INSPECTOR-HOURS ON SITE AND AT THE COMPANY OFFICES. INSPECTION WAS CONDUCTED IN THE AREAS OF LICENSEE ACTION ON PREVIOUS INSPECTION FINDINGS; QA PROGRAM - ANNUAL REVIEW; QA/QC ADMINISTRATION PROGRAM; ORGANIZATION AND ADMINISTRATION; DESIGN, DESIGN CHANGES AND MODIFICATIONS; DESIGN CHANGES AND MODIFICATIONS PROGRAM; TEST AND EXPERIMENTS PROGRAM; PROCUREMENT PROGRAM, RECEIPT, STORAGE AND HANDLING OF EQUIPMENT AND MATERIALS PROGRAM; RECORDS PROGRAM; DOCUMENT CONTROL PROGRAM; AUDIT PROGRAM; OFFSITE SUPPORT STAFF; HOUSEKEEPING/CLEANLINESS PROGRAM; LABORATORY QUALITY ASSURANCE PROGRAM; QA PERSONNEL ACCESS TO INFORMATION; HPCI AND RCIC OPERATIONAL FAILURES; REVIEW OF QA/QC INSPECTION PROGRAM; PLANT PROCEDURES REVIEW; AND ENVIRONMENTAL TECHNICAL SPECIFICATIONS. OF THE 20 AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN 12 AREAS; 12 ITEMS OF NONCOMPLIANCE WERE FOUND IN EIGHT AREAS (INFRACTION - FAILURE TO OBTAIN PROMPT CORRECTIVE ACTION; INFRACTION - FAILURE TO EXECUTE/ESTABLISH AN INSPECTION PROGRAM; INFRACTION - FAILURE TO REAUDIT DEFICIENT AREAS; INFRACTION - FAILURE TO RESPOND TO AUDIT FINDING REPORTS; INFRACTION - FAILURE TO CONTROL VENDOR MANUALS; INFRACTION - FAILURE TO IMPLEMENT PLANT PROCEDURES; INFRACTION - FAILURE TO HAVE APPROPRIATE QUANTITATIVE OR QUALITATIVE ACCEPTANCE CRITERIA; DEFICIENCY - FAILURE TO REVIEW STATUS AND ADEQUACY OF QA PROGRAM; DEFICIENCY - FAILURE TO REVIEW PLANT PROCEDURES; DEFICIENCY - FAILURE TO CONTROL CHANGES TO PURCHASE REQUISITIONS; DEFICIENCY - FAILURE TO MAINTAIN RECORDS; DEFICIENCY - FAILURE TO ESTABLISH MATERIALS STORAGE CONTROL.

INSPECTION JUNE 10 - JULY 14 (80-31): THIS INSPECTION INVOLVED 55 INSPECTOR-HOURS ON SITE IN THE AREA OF TECHNICAL SPECIFICATION COMPLIANCE, REPORTABLE OCCURRENCES, HOUSEKEEPING, OPERATOR PERFORMANCE, OVERALL PLANT OPERATIONS. QUALITY ASSURANCE PRACTICES, STATION AND CORPORATE PRACTICES, CORRECTIVE AND PREVENTATIVE MAINTENANCE ACTIVITIES, SITE SECURITY PROCEDURES, RADIATION CONTROL ACTIVITIES AND SURVEILLANCE ACTIVITIES. OF THE ELEVEN AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN NINE AREAS. TWO ITEMS OF NONCOMPLIANCE WERE FOUND IN TWO AREAS (INFRACTION - FAILURE TO PROPERLY ALIGN VALVES IN A SAFETY SYSTEM; INFRACTION - FAILURE TO CORRECT SEISMIC DEFICIENCIES IN A TIMELY MANNER.

INSPECTION JULY 30, 31 - AUGUST 4 (80-33): THIS SPECIAL, UNANNOUNCED INSPECTION INVOLVED 6 INSPECTOR-HOURS ON SITE AND AT THE GENERAL OFFICES IN THE AREA OF FOLLOWUP ON ITEMS RELATIVE TO 10 CFR PART 21. THE PREVIOUSLY IDENTIFIED ITEMS OF NONCOMPLIANCE IDENTIFIED IN IE REPORTS 50-321/79-33 AND 50-366/79-37 REMAIN OPEN.

INSPECTION AUGUST 13-14 (80-34): THIS INSPECTION INVOLVED SEVEN INSPECTOR-HOURS ON SITE BY ONE NRC INSPECTOR. THE INSPECTION WAS BEGUN DURING AN OFFSHIFT PERIOD; THREE INSPECTION HOURS WERE ACCOMPLISHED DURING OFFSHIFT PERIODS. AREAS INSPECTED INCLUDED REVIEW OF THE LICENSEE'S IMPLEMENTATION OF THE CONTINGENCY STRIKE PLAN. THE LICENSEE WAS FOUND TO BE IN COMPLIANCE WITH NRC REQUIREMENTS WITHIN THE AREAS EXAMINED DURING THE INSPECTION.

REPORTS RECEIVED FPOM LICENSEE

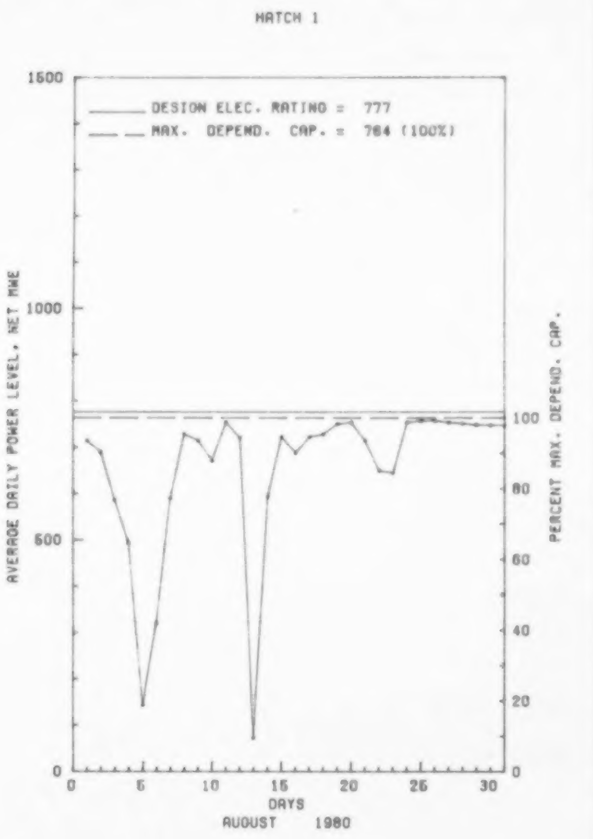
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-058/ 03L-0	06/16/80	07/08/80	CONTROL ROD ACCUMULATOR NITROGEN PRESSURE ABNORMAL
80-060/ 01L-0	06/03/80	07/01/80	MISSING HANGER SPRINKLER SYSTEM CABLE SPREADING ROOM
80-067/ 03L-0	06/28/80	07/17/80	"B" DIESEL TRIPPED DURING MANUAL START
80-070/ 03L-0	06/26/80	07/22/80	SNUBBER ON PIPING FOUND TO BE LEAKING OIL
80-075/ 03L-0	06/26/80	07/22/80	DRYWELL TEMPERATURE RECORDER PRINTING INTERMITTENTLY
80-076/ 03L-0	07/01/80	07/14/80	RESIDUAL HEAT REMOVAL SUPPORTS INADEQUATE FOR DESIGN BASIS LOADS
80-077/ 03L-0	07/08/80	07/22/80	REACTOR HIGH PRESSURE SWITCH SETPOINT DRIFT
80-078/ 03L-0	06/30/80	07/14/80	PRIMARY CONTAINMENT OXYGEN CONCENTRATION EXCEEDS TECH SPECS REQUIREMENTS
80-079/ 03L-0	07/02/80	07/14/80	SMOKE DETECTORS IN SEVEN PLANT AREAS NOT TESTED AS REQUIRED BY TECHNICAL SPECIFICATIONS
80-081/ 03L-0	07/05/80	07/22/80	DRYWELL PRESSURE RECORDER INOPERABLE
80-082/ 03L-0	7/11/80	07/30/80	HIGH AMBIENT TEMPERATURE IN LOW PRESSURE COOLANT INJECTION ROOM
80-083/ 03L-0	07/13/80	08/05/80	MISSED SURVEILLANCE TEST FOR DRYWELL PRESSURE SWITCH
80-085/ 03L-0	07/20/80	08/07/80	REACTOR WATER SAMPLE MISSED
80-086/ 01T-0	07/25/80	08/04/80	SUPPORT MISSING ON INSTRUMENT AIR SYSTEM PIPE
80-087/ 01T-0	07/24/80	08/05/80	REACTOR CORE ISOLATION COOLING DISCHARGE LINE ANCHOR SUPPORT
80-091/ 03L-0	07/24/80	08/12/79	CONTROL ROOM AIR CHLORINE FAN MOTOR INOPERABLE

REVIEWED BY OMPA LOEB: DATE: 10/6/80

REVIEWED BY NRR DL: DATE: 10/7/80

HATCH 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-321
2. Utility Contact: R.M. BLACKWELL (912) 367-7781
3. Licensed Thermal Power (MWt): 2436
4. Nameplate Rating (Gross MWe): 1000 x 0.85 = 850
5. Design Electrical Rating (Net MWe): 777
6. Maximum Dependable Capacity (Gross MWe): 797
7. Maximum Dependable Capacity (Net MWe): 764
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	5,855.0	47,419.0
12. Hours Reactor Was Critical	722.9	4,889.8	30,687.8
13. Reactor Reserve Shutdown Hours	0	0	0
14. Hours Generator On-Line	708.2	4,460.1	28,228.5
15. Unit Reserve Shutdown Hours	0	0	0
16. Gross Thermal Energy (MWH)	1,587,689	10,012,853	60,365,888
17. Gross Electrical Energy (MWH)	507,230	3,275,770	19,566,710
18. Net Electrical Energy (MWH)	484,603	3,119,762	18,581,695
19. Unit Service Factor	95.2	76.2	70.2
20. Unit Availability Factor	95.2	76.2	70.2
21. Unit Capacity Factor (MDC Net)	85.3	69.7	59.4
22. Unit Capacity Factor (DER Net)	83.8	68.6	58.4
23. Unit Forced Outage Rate	4.8	23.8	17.4

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-34	08/02/80	F	20.7	H	5		RB	CONROD	REDUCE LOAD IN ORDER TO PERFORM ROD PATTERN ADJUSTMENT.
80-35	08/04/80	F	18.7	G	3		EC	INSTRU	OPERATOR INADVERTENTLY REPLACED A 6-WATT 145 VOLT INDICATING LIGHT IN THE CONTROL PANEL WITH A 6 WATT 6 VOLT INDICATING LIGHT THUS CAUSING A 3-AMP FUSE IN EHC CABINET TO BLOW.
80-36	08/05/80	F	4.4	H	5		EC	INSTRU	UNIT ON STARTUP RAMP FROM ABOVE SHUTDOWN #80-35.
80-37	08/05/80	F	26.5	H	5		HC	HTEXCH	REDUCED LOAD TO INCREASE CONDENSER VACUUM.
80-38	08/06/80	F	15.8	H	5		HC	HTEXCH	REDUCED POWER TO SWAP OUT SJAES.
80-39	08/09/80	F	9.5	H	5		RB	CONROD	REDUCE LOAD IN ORDER TO PERFORM ROD PATTERN ADJUSTMENT.
80-40	08/12/80	F	17.1	A	3		CH	PUMPXX	LOW RX WATER LEVEL BECAUSE OF LOSS OF FEED PUMP.
80-41	08/13/80	F	21.0	H	5		CH	PUMPXX	UNIT ON STARTUP RAMP FROM ABOVE SHUTDOWN #80-40.
80-42	08/16/80	F	5.6	H	5		RB	CONROD	REDUCE LOAD IN ORDER TO PERFORM ROD PATTERN ADJUSTMENT.
80-43	08/18/80	F	3.5	A	5		WA	VALVEX	PSW TO H2 COOLER 4-WAY VALVE WAS NOT PROPERLY ALIGNED TO ALLOW TOTAL PSW TO PASS THROUGH COOLER.
80-44	08/21/80	F	6.0	A	5		HC	HTEXCH	REDUCED LOAD TO INCREASE CONDENSER VACUUM. LOST VACUUM BECAUSE OF VALVE N62-F038 STUCK CLOSED.
80-45	08/22/80	F	11.7	A	3	80-103	WA	FILTER	LOSS OF PSW BECAUSE OF SHEARED KEY-WAY IN DIVISION II STRAINERS. KEY-WAY SHEARED BECAUSE OF DEBRIS.

SUMMARY

HATCH I OPERATED AT ABOUT 90% FOR THE MONTH, WITH TWO OUTAGES AND NUMEROUS REDUCTIONS LISTED ABOVE.

LEGEND

[1] Type:
- F–Forced
- S–Scheduled

[2] Reason:
- A–Equipment Failure
- B–Maintenance or Test
- C–Refueling
- D–Regulatory Restriction
- E–Operator Training & License Examination
- F–Administrative
- G–Operational Error
- H–Other

[3] Method:
- 1–Manual
- 2–Manual Scram
- 3–Automatic Scram
- 4–Continuations
- 5–Load Reduction
- 9–Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

HATCH 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Georgia/Appling 11 miles N of Baxley
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2436
5.	Design Electrical Rating (MWE Net)	784
6.	Date of Initial Criticality	July 4, 1978
7.	Date of Commercial Operation	September 5, 1979
8.	Condenser Cooling Method	Cooling Towers
9.	Condenser Cooling Water Source	Altamaha River

Utility & Contractor Information

10.	Licensee	Georgia Power Company
11.	Corporate Address	270 Peach Street, NE Atlanta, Georgia 30303
12.	Corporate Contact	J. H. Miller Executive Vice President
13.	Architect/Engineer	Southern Services, Inc./Bechtel
14.	Nuclear Steam Supply System	GE
15.	Constructor	Georgia Power Company

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. Lewis
18.	IE Section Chief	H. C. Dance
19.	IE Resident Inspector	R. F. Rogers (912) 367 9881
20.	Licensing Project Manager	D. Verrelli
21.	Docket Number	50-366
22.	License Number/Date of Issue	NPF-5/June 13, 1978
23.	Public Document Room	Appling County Public Library Parker Street Baxley, Georgia 31513

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

PLANT IS OPERATING AT FULL POWER.

LAST IE SITE INSPECTION DATE: AUGUST 13-14, 1980

INSPECTION REPORT NO: 50-366/80-34

SUMMARY

INSPECTION SUMMARY

INSPECTION JULY 7-11, 14-18, 22-24 (80-28): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 144 INSPECTOR-HOURS ON SITE AND AT THE COMPANY OFFICES. INSPECTION WAS CONDUCTED IN THE AREAS OF LICENSEE ACTION ON PREVIOUS INSPECTION FINDINGS; QA PROGRAM - ANNUAL REVIEW; QA/QC ADMINISTRATION PROGRAM; ORGANIZATION AND ADMINISTRATION; DESIGN, DESIGN CHANGES AND MODIFICATIONS; DESIGN CHANGES AND MODIFICATIONS PROGRAM; TEST AND EXPERIMENTS PROGRAM; PROCUREMENT PROGRAM, RECEIPT, STORAGE AND HANDLING OF EQUIPMENT AND MATERIALS PROGRAM, RECORDS PROGRAM; DOCUMENT CONTROL PROGRAM; AUDIT PROGRAM; OFFSITE SUPPORT STAFF; HOUSEKEEPING/CLEANLINESS PROGRAM; LABORATORY QUALITY ASSURANCE PROGRAM; QA PERSONNEL ACCESS TO INFORMATION; HPCI AND RCIC OPERATIONAL FAILURES; REVIEW OF QA/QC INSPECTION PROGRAM; PLANT PROCEDURES REVIEW; AND ENVIRONMENTAL TECHNICAL SPECIFICATIONS. OF THE 20 AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN 12 AREAS, 12 ITEMS OF NONCOMPLIANCE WERE FOUND IN EIGHT AREAS (INFRACTION - FAILURE TO OBTAIN PROMPT CORRECTIVE ACTION; INFRACTION - FAILURE TO EXECUTE/ESTABLISH AN INSPECTION PROGRAM; INFRACTION - FAILURE TO REAUDIT DEFICIENT AREAS; INFRACTION - FAILURE TO RESPOND TO AUDIT FINDING REPORTS; INFRACTION - FAILURE TO CONTROL VENDOR MANUALS; INFRACTION - FAILURE TO IMPLEMENT PLANT PROCEDURES; INFRACTION - FAILURE TO HAVE APPROPRIATE QUANTITATIVE OR QUALITATIVE ACCEPTANCE CRITERIA; DEFICIENCY - FAILURE TO REVIEW STATUS AND ADEQUACY OF QA PROGRAM; DEFICIENCY - FAILURE TO REVIEW PLANT PROCEDURES; DEFICIENCY - FAILURE TO CONTROL CHANGES TO PURCHASE REQUISITIONS; DEFICIENCY - FAILURE TO MAINTAIN RECORDS; DEFICIENCY - FAILURE TO ESTABLISH MATERIALS STORAGE CONTROL.

INSPECTION JUNE 10 - JULY 14 (80-31): THIS INSPECTION INVOLVED 55 INSPECTOR-HOURS ON SITE IN THE AREA OF TECHNICAL SPECIFICATION COMPLIANCE, REPORTABLE OCCURRENCES, HOUSEKEEPING, OPERATOR PERFORMANCE, OVERALL PLANT OPERATIONS. QUALITY ASSURANCE PRACTICES, STATION AND CORPORATE PRACTICES, CORRECTIVE AND PREVENTATIVE MAINTENANCE ACTIVITIES, SITE SECURITY PROCEDURES, RADIATION CONTROL ACTIVITIES AND SURVEILLANCE ACTIVITIES. OF THE ELEVEN AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN NINE AREAS. TWO ITEMS OF NONCOMPLIANCE WERE FOUND IN TWO AREAS (INFRACTION - FAILURE TO PROPERLY ALIGN VALVES IN A SAFETY SYSTEM; INFRACTION - FAILURE TO CORRECT SEISMIC DEFICIENCIES IN A TIMELY MANNER.

INSPECTION JULY 30, 31 - AUGUST 4 (80-33): THIS SPECIAL, UNANNOUNCED INSPECTION INVOLVED 6 INSPECTOR-HOURS ON SITE AND AT THE GENERAL OFFICES IN THE AREA OF FOLLOWUP ON ITEMS RELATIVE TO 10 CFR PART 21. THE PREVIOUSLY IDENTIFIED ITEMS OF NONCOMPLIANCE IDENTIFIED IN IE REPORTS 50-321/79-33 AND 50-366/79-37 REMAIN OPEN.

INSPECTION AUGUST 13-14 (80-34): THIS INSPECTION INVOLVED SEVEN INSPECTOR-HOURS ON SITE BY ONE NRC INSPECTOR. THE INSPECTION WAS BEGUN DURING AN OFFSHIFT PERIOD; THREE INSPECTION HOURS WERE ACCOMPLISHED DURING OFFSHIFT PERIODS. AREAS INSPECTED INCLUDED REVIEW OF THE LICENSEE'S IMPLEMENTATION OF THE CONTINGENCY STRIKE PLAN. THE LICENSEE WAS FOUND TO BE IN COMPLIANCE WITH NRC REQUIREMENTS WITHIN THE AREAS EXAMINED DURING THE INSPECTION.

REPORTS RECEIVED FROM LICENSEE

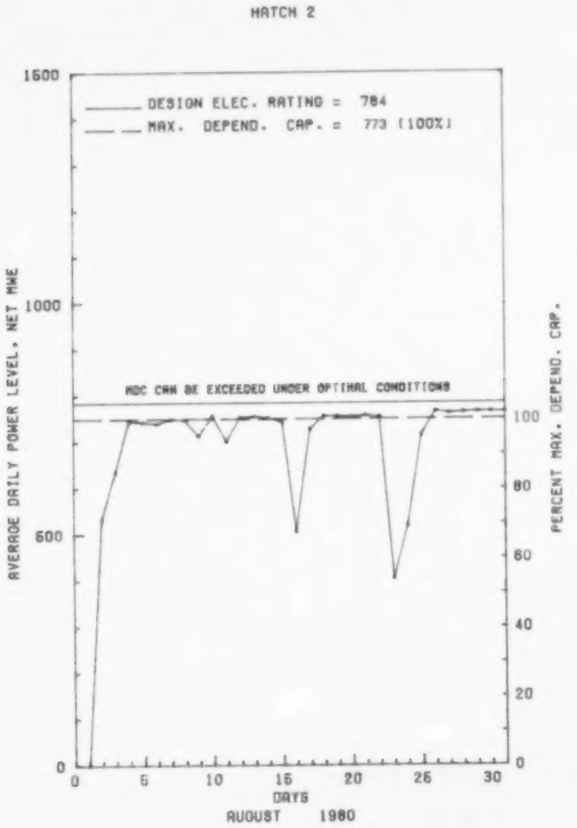
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-005/03L-0	01/19/80	02/07/80	TECHNICAL SPECIFICATION DELAYED REQUIRED ACTION
80-062/01T-0	06/12/80	06/25/80	VACUUM BREAKER FAILED TO OPEN
80-097/03L-0	06/25/80	07/15/80	HIGH PRESSURE COOLANT INJECTION STEAM LINE FLOW INSTRUMENT SET POINT DRIFT
80-098/03L-0	06/26/80	07/14/80	HIGH PRESSURE COOLANT INJECTION SUCTION PRESSURE TRANSMITTER SET POINT DRIFT
80-099/03L-0	06/26/80	07/17/80	"B" DRYWELL HYDROGEN - OXYGEN ANALYZER REAGENT GAS FLOW IMPROPER
80-101/03L-0	07/11/80	07/29/80	HIGH PRESSURE COOLANT INJECTION SYSTEM MOTOR OPERATED VALVE MOTOR WINDING FAILURE
80-102/03L-0	07/11/80	08/05/80	MISSED SURVEILLANCE TEST FOR REACTOR PROTECTION SYSTEM MG-SET
80-103/03L-0	07/09/80	07/17/80	HIGH DRYWELL PRESSURE SWITCHES SET POINT DRIFT
80-104/03L-0	07/11/80	08/05/80	4KV 2F NORMAL SUPPLY BREAKER TRIPPED
80-107/03L-0	07/19/80	08/04/80	METHOD FOR VERIFYING JET PUMP OPERABILITY NOT IN FULL COMPLIANCE WITH TECH SPECS
80-108/03L-0	07/20/80	08/07/80	TORUS - DRYWELL VACUUM BREAK FAILED TO OPEN DURING TESTING
80-110/03L-0	07/22/80	08/04/80	SCRAM DISCHARGE ROD BLOCK LEVEL SWITCH FOUND ISOLATED

REVIEWED BY OMPA LOEB: **DATE:** 10/6/80

REVIEWED BY NRR DL: **DATE:** 10/7/80

HATCH 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-366
2. Utility Contact: R.M. BLACKWELL (912) 367-7781
3. Licensed Thermal Power (MWt): 2436
4. Nameplate Rating (Gross MWe): 1000 x .85 = 850
5. Design Electrical Rating (Net MWe): 784
6. Maximum Dependable Capacity (Gross MWe): 806
7. Maximum Dependable Capacity (Net MWe): 773
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	5,855.0	8,688.0
12. Hours Reactor Was Critical	731.6	4,190.1	6,853.6
13. Reactor Reserve Shutdown Hours	0	0	0
14. Hours Generator On-Line	722.5	3,937.3	6,352.0
15. Unit Reserve Shutdown Hours	0	0	0
16. Gross Thermal Energy (MWH)	1,625,414	8,439,894	13,133,156
17. Gross Electrical Energy (MWH)	533,900	2,866,730	4,702,690
18. Net Electrical Energy (MWH)	510,506	2,730,439	4,487,576
19. Unit Service Factor	97.1	67.2	73.1
20. Unit Availability Factor	97.1	67.2	73.1
21. Unit Capacity Factor (MDC Net)	88.8	60.3	66.8
22. Unit Capacity Factor (DER Net)	87.5	59.5	65.9
23. Unit Forced Outage Rate	2.9	13.5	11.3

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-26	08/01/80	F	21.5	D	2		IC	ACCUMU	RX MANUAL SCRAM PER IEB 80-17 SDV ANALYSIS PROVED SATISFACTORY.
80-27	08/01/80	F	19.5	B	5		IC	ACCUMU	UNIT ON STARTUP RAMP FROM ABOVE SHUTDOWN #80-26.
80-28	08/03/80	F	15.6	B	5		HC	HTEXCH	REDUCED LOAD IN ORDER TO ADJUST 4TH AND 6TH STAGE FEEDWATER HEATER LEVEL CONTROLLERS.
80-29	08/08/80	F	5.5	B	5		HE	TURBIN	REDUCED LOAD IN ORDER TO PERFORM WEEKLY TURBINE TEST.
80-30	08/11/80	F	4.2	A	5	80-120	CB	PUMPXX	2B RECIRC PUMP MG SET TRIP ON LOW OIL PRESSURE REDUCED LOAD TO REDUCED SPEED ON 2A RECIRC PUMP.
80-31	08/15/80	F	26.4	A	5	80-123	CE	VALVEX	REDUCED LOAD TO REPAIR TORQUE SWITCH ON E51-F008 VALVE IN RCIC SYSTEM.
80-32	08/23/80	F	2.0	B	5		HE	TURBIN	REDUCED LOAD IN ORDER TO PERFORM WEEKLY TURBINE TEST.
80-33	08/23/80	F	46.2	B	5		RB	CONROD	REDUCED LOAD IN ORDER TO PERFORM A ROD SEQUENCE EXCHANGE.

SUMMARY

HATCH II OPERATED AT ABOUT 90% POWER FOR THE MONTH, WITH ONE OUTAGE AND NUMEROUS REDUCTIONS LISTED ABOVE.

LEGEND

[1] Type:
F-Forced
S-Scheduled

[2] Reason:
A-Equipment Failure
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error
H-Other

[3] Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Continuations
5-Load Reduction
9-Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

HUMBOLDT BAY

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	California/Humboldt 4 miles S of Eureka
2.	Electric Reliability Council	Western Systems Coordinating Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	220
5.	Design Electrical Rating (MWE Net)	65
6.	Date of Initial Criticality	February 16, 1963
7.	Date of Commercial Operation	August 1963
8.	Condenser Cooling Method	Circulating sea water
9.	Condenser Cooling Water Source	Pacific Ocean (Humboldt Bay)

Utility & Contractor Information

10.	Licensee	Pacific Gas & Electric Co.
11.	Corporate Address	77 Beale Street San Francisco, California 94106
12.	Corporate Contact	P. A. Crane, Jr. Asst. General Counsel
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	GE
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsibl	V
17.	IE Branch Chief	J. L. Crews
18.	IE Section Chief	D. M. Sternberg
19.	IE Resident Inspector	J. D. Carlson
20.	Licensing Project Manager	V. L. Rooney
21.	Docket Number	50-133
22.	License Number/Date of Issue	DPR-7, 8/28/62
23.	Public Document Room	Humboldt County Library 636 F Street Eureka, California 95501

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

THE PLANT HAS BEEN SHUT DOWN SINCE JULY 1976.

ITEMS REMAINING TO BE COMPLETED DURING THE OUTAGE HAVE BEEN DEFERRED PENDING RESOLUTION OF THE SEISMIC ISSUES AT THE PLANT SITE. A DATE FOR RETURN TO OPERATIONS HAS NOT BEEN ESTABLISHED.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

PLANT IS SHUTDOWN.

LAST IE SITE INSPECTION DATE: 05/20-22/80

INSPECTION REPORT NO: 50-133/80-02

SUMMARY

INSPECTION SUMMARY

NO INSPECTIONS CONDUCTED IN AUGUST 1980.

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-03/03L-0	07-11-80	08-10-80	FAILURE OF GE TYPE AK-1-15 CIRCUIT BREAKERS

LICENSE STATUS

Order issued 5/21/76 requiring Utility to prove seismic qualification before resuming operation of unit after outage of 7/2/76.

REVIEWED BY OMPA LOEB: John L Crooks **DATE:** 10/6/80

REVIEWED BY NRR DL: **DATE:** 10/1/80

HUMBOLDT BAY

AVERAGE DAILY POWER LEVEL (MWe)

OPERATING STATUS

1. Docket Number: 50-133
2. Utility Data Prepared By: B. C. Getty (707) 443-2787
3. Licensed Thermal Power (MWt): 220
4. Nameplate Rating (Gross MWe): 77 X 0.85 = 65
5. Design Electrical Rating (Net MWe): 65
6. Maximum Dependable Capacity (Gross MWe): 0
7. Maximum Dependable Capacity (Net MWe): 0
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons: None

9. Power Level To Which Restricted, If Any (Net MWe): None
10. Reasons For Restrictions, If Any

This M ive

11. Hours In Reporting Period
12. Number Of Hours Reactor Was Cri
13. Reactor Reserve Shutdow
14. Hours Generato
15. Unit R
16. rd)
erated (MWH)
Net gy Generated (MWH)
19. Unit S ce Factor
20. Unit Availability Factor
21. Unit Capacity Factor (Using MDC Net) 1/
22. Unit Capacity Factor (Using DER Net) 1/
23. Unit Forced Outage Rate
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): None

25. If Shut Down At End Of Report Period, Estimated Date of Startup: UNKNOWN

NO DECISION ON FUTURE OPERATION
SHUTDOWN INDEFINITELY

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
SEE SUMMARY BELOW									

SUMMARY

THE UNIT WAS SHUTDOWN ON 7/2/76 FOR SEISMIC MODIFICATIONS. THE UNIT REMAINS IN THE COLD SHUTDOWN CONDITION.

LEGEND

[1] Type:
F-Forced
S-Scheduled

[2] Reason:
A-Equipment Failure
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error
H-Other

[3] Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Continuations
5-Load Reduction
9-Other

[4] **Exhibit F:**
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] **Exhibit H:**
Same source.

INDIAN POINT 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	New York/Westchester 25 miles N of New York City
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2758
5.	Design Electrical Rating (MWE Net)	873
6.	Date of Initial Criticality	May 22, 1973
7.	Date of Commercial Operation	August 1973
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Hudson River

Utility & Contractor Information

10.	Licensee	Consolidated Edison Company of New York, Inc.
11.	Corporate Address	4 Irving Place New York, New York
12.	Corporate Contact	P. Zarakas Vice President
13.	Architect/Engineer	UE&C
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	J. A. Jones/UE&C

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Senior Inspectc	H. B. KISTER
19.	IE Resident Inspector	T. Rebelowski (914-739-9360) FTS: 264-3311
20.	Licensing Project Manager	L. Olshan (X27144)
21.	Docket Number	50-247
22.	License Number/Date of Issue	DPR-26/9-28-73
23.	Public Document Room	White Plains Public Library 100 Martine Avenue White Plains, New York 10601

INSPECTION STATUS

ENFORCEMENT STATUS

FAILURE TO REQUALIFY THREE SECURITY OFFICERS.
(8235 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

- THE UNIT HAS EXPERIENCED CONDENSER TUBE LEAKS THAT HAVE REQUIRED TEMPORARY REDUCTIONS IN POWER.

FACILITY ITEMS (PLANS AND PROCEDURES):

- LICENSEE REFUELING OUTAGE IS SCHEDULED FOR LATE DECEMBER.

PLANT STATUS:

- Unit at 100% power. (790 MWE)

- LAST IE SITE INSPECTION DATE: 8/18-22/80
- INSPECTION REPORT NO: 50-247/80-13

SUMMARY

INSPECTION SUMMARY

- 50-247/80-02 - MAR 10-21: SPECIAL, ANNOUNCED HEALTH PHYSICS APPRAISAL INCLUDING A REVIEW OF SELECTED PROCEDURES AND REPRESENTATIVE RECORDS, OBSERVED WORK PRACTICES AND INTERVIEWED PERSONNEL. ONE ITEM OF NONCOMPLIANCE WAS IDENTIFIED: FAILURE TO PERFORM A WRITTEN SAFETY EVALUATION IN ACCORDANCE WITH REGULATORY REQUIREMENT 10 CFR 50.59.

REPORTS RECEIVED FROM LICENSEE

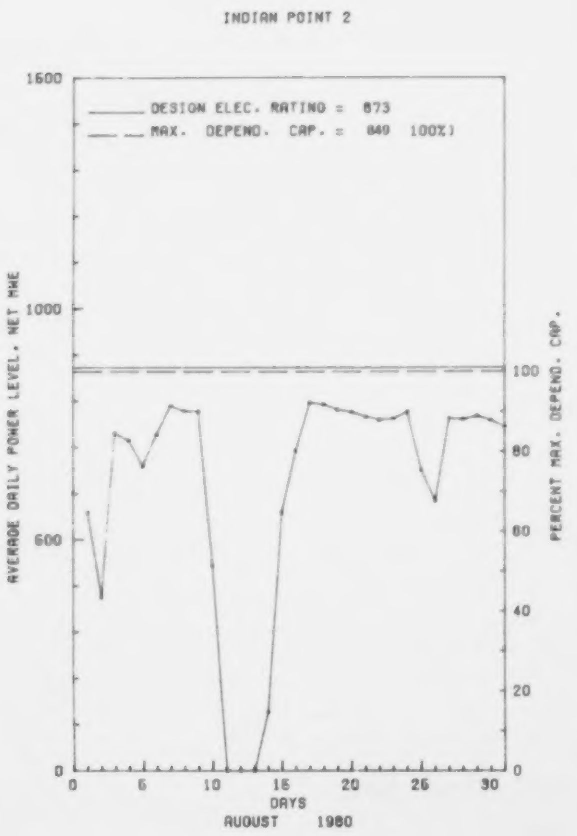
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-007/03L	06/21/80	07/17/80	NO. 23 AUX. FEEDWATER PUMP HAD BEEN TRANSFERRED TO LOCAL. THE SWITCH WAS RESTORED TO THE CONTROL ROOM WITH THE TIME PERIOD SPECIFIED IN TECH. SPECS.
80-008/03L	07/01/80	07/30/80	DURING NORMAL OPERATION VALVE 863 N2 SUPPLY WAS AT FAULT. A PLANT SHUTDOWN WAS COMMENCED LATER HALTED AFTER REPAIR OF N2 SUPPLY.

REVIEWED BY OMPA LOEB: T. C. Cintula **DATE:** 10·6·80

REVIEWED BY NRR DL: **DATE:** 10-7-80

INDIAN POINT 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-247
2. Utility Contact: E. EICH (914) 694-6000 X I.P.
3. Licensed Thermal Power (MWt): 2758
4. Nameplate Rating (Gross MWe): 1126 x 0.9 = 1013
5. Design Electrical Rating (Net MWe): 873
6. Maximum Dependable Capacity (Gross MWe): 885
7. Maximum Dependable Capacity (Net MWe): 849

ITEM 21 CALCULATED BY USING A WEIGHTED AVERAGE

8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): 824
10. Reasons for Restrictions, If Any: REMOVAL OF #3 DISC. (GEN. END) ON #23 TURBINE ROTOR.

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	5,855.0	54,096.0
12. Hours Reactor Was Critical	706.9	4,754.9	37,069.4
13. Reactor Reserve Shutdown Hours	31.1	883.5	1,126.9
14. Hours Generator On-Line	642.4	4,582.9	35,972.1
15. Unit Reserve Shutdown Hours	0	.0	.5
16. Gross Thermal Energy (MWH)	1,665,327	12,058,174	92,590,700
17. Gross Electrical Energy (MWH)	482,540	3,630,660	28,763,746
18. Net Electrical Energy (MWH)	458,442	3,460,395	27,425,495
19. Unit Service Factor	86.3	78.3	66.5
20. Unit Availability Factor	86.3	78.3	66.5
*21. Unit Capacity Factor (MDC Net)	72.6	69.6	58.9
22. Unit Capacity Factor (DER Net)	70.6	67.7	58.1
23. Unit Forced Outage Rate	13.7	10.8	8.4

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): CYCLE 4/5 REFUELING OUTAGE - 12/31/80 - 3 MONTHS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

*COMPUTED SINCE 7/1/74, THE DATE OF COMPLETION OF A 100 DAY – 100% POWER OPERATION TEST.

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
12	08/01/80	F	11.7	A	3		CH	PUMPXX	LOW LEVEL IN NO. 23 S/G CAUSED BY TRIP ON NO. 22 MBFP.
13	08/10/80	F	89.9	A	2		HA	HTEXCH	REPAIR MAIN TURBINE OIL COOLER.

SUMMARY

INDIAN POINT 2 OPERATED WITH ONE OUTAGE FOR STEAM GENERATOR PUMP PROBLEMS AND ANOTHER OUTAGE TO REPAIR THE MAIN TURBINE OIL COOLER.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

INDIAN POINT 3

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	New York/Westchester 25 miles N of New York City
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	3025
5.	Design Electrical Rating (MWE Net)	965
6.	Date of Initial Criticality	April 6, 1976
7.	Date of Commercial Operation	August 30, 1976
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Hudson River

Utility & Contractor Information

10.	Licensee	Power Authority of the State of New York, Inc.
11.	Corporate Address	10 Columbus Circle New York, New York
12.	Corporate Contact	J. P. Bayne - Senior Vice President
13.	Architect/Engineer	UE&C
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	WEDCO

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Section Chief	H. B. KISTER
19.	IE Resident Inspector	T. Rebelowski (914-739-9360) FTS: 264-3311 - Operator
20.	Licensing Project Manager	L. Olshan x27144
21.	Docket Number	50-286
22.	License Number/Date of Issue	DPR-64, 12/12/75
23.	Public Document Room	White Plains Public Library 100 Martine Avenue White Plains, New York 10601

INSPECTION STATUS

ENFORCEMENT STATUS

10 CFR 20.101(A) LIMITS THE TOTAL OCCUPATIONAL DOSE RECEIVED PER CALENDAR QUARTER BY INDIVIDUALS IN RESTRICTED AREAS TO 18 3/4 REMS TO THE EXTREMITIES, AND 7 1/2 REMS TO THE SKIN OF THE WHOLE BODY. IN ADDITION, IF CERTAIN SPECIFIED CONDITIONS ARE MET, SECTION 20.101(B) ALLOWS WHOLE BODY EXPOSURES OF 3 REMS PER CALENDAR QUARTER. CONTRARY TO THE ABOVE, DURING THE SECOND CALENDAR QUARTER OF 1980 (APRIL-JUNE), TWO INDIVIDUALS SUSTAINED WHOLE BODY EXPOSURES OF 4.209 REMS AND 4.130 REMS, RESPECTIVELY. ADDITIONALLY, ONE OF THE INDIVIDUALS SUSTAINED EXPOSURE OF THE SKIN TO 8.180 REMS AND THE OTHER SUSTAINED AN EXTREMITY EXPOSURE OF 43.569 REMS.
(8009 INFRACTION)

10CFR 20.202 REQUIRES THAT EACH LICENSEE SUPPLY APPROPRIATE PERSONNEL MONITORING EQUIPMENT AND REQUIRE ITS USE BY EACH INDIVIDUAL WHO ENTERS A RESTRICTED AREA UNDER SUCH CIRCUMSTANCES THAT HE RECEIVES, OR IS LIKELY TO RECEIVE, A DOSE IN ANY CALENDAR QUARTER IN EXCESS OF 25 PERCENT OF THE APPLICABLE VALUE SPECIFIED IN PARAGRAPH (A) OF SECTION 20.101. CONTRARY TO THE ABOVE, ON JUNE 24, 1980 DURING REPAIR OF INCORE FLUX DETECTOR DRIVE COMPONENTS, TWO INDIVIDUALS RECEIVED RADIATION DOSES TO THE EXTREMITIES IN EXCESS OF 25 PERCENT OF THE VALUE SPECIFIED IN 10 CFR 20.201(A) AND APPROPRIATE EXTRIMITY MONITORING EQUIPMENT WAS NOT SUPPLIED.
(8009 INFRACTION)

10 CFR 50.72 REQUIRES THAT EACH LICENSEE NOTIFY THE NRC OPERATIONS CENTER AS SOON AS POSSIBLE AND IN ALL CASES WITHIN ONE HOUR OF THE OCCURRENCE OF ANY EVENT MEETING THE CRITERIA OF 10 CFR 20.403. 10 CFR 20.403 REQUIRES EACH LICENSEE TO NOTIFY THE NRC OF ANY INCIDENT INVOLVING LICENSED MATERIAL WHICH MAY HAVE CAUSED OR THREATENS TO CAUSE EXPOSURE OF THE WHOLE BODY OF ANY INDIVIDUAL TO 5 REMS OR MORE OF RADIATION. CONTRARY TO THE ABOVE, ON JUNE 24, 1980, DURING AN INCIDENT INVOLVING WORK ON INCORE FLUX DETECTOR COMPONENTS, THREE INDIVIDUALS WERE EXPOSED TO RADIATION WHICH MAY HAVE CAUSED WHOLE BODY EXPOSURES IN EXCESS OF 5 REMS AND THE NRC OPERATIONS CENTER WAS NOT NOTIFIED.
(8009 INFRACTION)

TECHNICAL SPECIFICATION 6.8 REQUIRES THAT WRITTEN PROCEDURES BE ESTABLISHED, IMPLEMENTED AND MAINTAINED COVERING THE ACTIVITIES RECOMMENDED IN APPENDIX A OF REGULATORY GUIDE 1.33, NOVEMBER 1972. THIS REGULATORY GUIDE REQUIRES THAT PROCEDURES FOR REPAIR OR REPLACEMENT OF EQUIPMENT BE PREPARED PRIOR TO BEGINNING WORK. CONTRARY TO THE ABOVE, AS OF JUNE 24, 1980, REPAIR OF THE INCORE FLUX MONITORING SYSTEM AND REPLACEMENT OF NEUTRON DETECTORS HAD BEEN CONDUCTED WITHOUT PREPARATION AND USE OF THE REQUIRED PROCEDURES.
(8009 INFRACTION)

TECHNICAL SPECIFICATION 6.11 REQUIRES THAT PROCEDURES FOR PERSONNEL RADIATION PROTECTION SHALL BE PREPARED CONSISTENT WITH THE REQUIREMENTS OF 10 CFR 20 AND SHALL BE FOLLOWED IN ALL OPERATIONS INVOLVING PERSONNEL RADIATION EXPOSURE. RADIOLOGICAL AND ENVIRONMENTAL SERVICES PROCEDURE NO. RE-HP-P204, "CONTAINMENT ENTRY," REQUIRES IN SECTION 7 THAT A "CONDITION AT TIME OF CONTAINMENT ENTRY FORM" BE COMPLETED PRIOR TO CONTAINMENT ENTRY. CONTRARY TO THE ABOVE, THE "FRACTIONAL PART OF MPC IN CONTAINMENT" AND "AREAS TO BE SURVEYED" SECTIONS OF THE ENTRY FORM WERE NOT COMPLETED FOR A CONTAINMENT ENTRY MADE BY PERSONNEL TO REPAIR AN INCORE FLUX DETECTOR DRIVE ON JUNE 24, 1980.
(8009 DEFICIENCY)

RADIOLOGICAL AND ENVIRONMENTAL SERVICES PROCEDURE NO. RE-HPP-4.1, "ISSUANCE AND CONTROL OF PERSONNEL DOSIMETRY," REQUIRES IN SECTION 7.1.5.2 THAT EXTREMITY DOSIMETRY SHOULD BE ISSUED WHEN IT IS LIKELY THAT THE EXTREMITY DOSE WILL EXCEED 5 TIMES THE WHOLE BODY EXPOSURE AND BE GREATER THAN 500 MREM/HR TO THE EXTREMITY. THIS PROCEDURE FOR PERSONNEL RADIATION PROTECTION IS NOT CONSISTENT WITH THE REQUIREMENTS OF 10 CFR 20.202, "PERSONNEL MONITORING," IN THAT IT DOES NOT PROVIDE ADEQUATE GUIDANCE FOR EXTREMITY MONITORING IF PERSONNEL EXPOSURES IN ANY CALENDAR QUARTER ARE LIKELY TO EXCEED 25 PERCENT OF THE APPLICABLE VALUE SPECIFIED IN PARAGRAPH (A) OF SECTION 20.101.
(8009 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

FACILITY ITEMS (PLANS AND PROCEDURES):

SIX WEEK OUTAGE IS PLANNED FOR OCTOBER 1 TO NOVEMBER 15, 1980 AND STEAM GENERATOR INSPECTIONS.

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-010/03L	06/29/80	07/28/80	AFTER OVERSPEED TRIP TEST, THE OPERATOR WAS UNABLE TO RESET THE TRIP LATCH LEVER ON D/G 32. DIESEL DECLARED INOPERABLE AT 1330 ON JUNE 29; RETURNED TO SERVICE AT 1850 ON JUNE 29.
80-011/03L	07/16/80	08/12/80	NO. 31 CONTAINMENT FAN MOTOR COOLER LEAKING.
80-012/03L	07/16/80	08/15/80	PIPING LEAK VALVE C-107 - CHARGING LINE TO RHX

MANAGERIAL ITEMS:

A MANAGEMENT INSPECTION WAS CONDUCTED AT CORPORATE HEADQUARTERS FROM AUGUST 13 TO 15, 1980.

FACILITY ITEMS (PLANS AND PROCEDURES):

NRR MET WITH THE LICENSEE REGARDING ENVIRONMENTAL QUALIFICATIONS OF ELECTRICAL EQUIPMENT.

PLANT STATUS:

Unit at 100% power.

LAST IE SITE INSPECTION DATE: 8/13-15/80

INSPECTION REPORT NO: 50-286/80-12

SUMMARY

INSPECTION SUMMARY

50-286/80-05 - MAR 15 - APR 26: ROUTINE ONSITE AND REGULAR BACKSHIFT INSPECTIONS BY THE RESIDENT INSPECTOR (46 HRS) OF PLANT OPERATIONS INCLUDING SHIFT LOGS AND RECORDS; LICENSEE ACTION ON PREVIOUS INSPECTION FINDINGS; OBSERVATION OF PHYSICAL SECURITY; REVIEW OF MONTHLY AND PERIODIC REPORTS; OPERATIONAL PROBLEMS; SPURIOUS SAFETY INJECTION, IAL REVIEW, REVIEW AND VERIFICATION OF INTERIM MEASURES REQUIRED BY THE NRC CONFIRMATORY ORDER DATED FEBRUARY 11, 1980 - SIXTY DAY REQUIREMENTS. NO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED.

50-286/80-07 - APR 26 - JUN 7: ROUTINE ONSITE, REGULAR AND BACKSHIFT INSPECTIONS BY TWO RESIDENT INSPECTORS (43 HRS) OF PLANT OPERATIONS INCLUDING SHIFT LOGS AND RECORDS; (LICENSEE ACTION ON PREVIOUS INSPECTION FINDINGS); OBSERVATION OF PHYSICAL SECURITY; OPERATIONAL PROBLEMS, SCHEDULED EMERGENCY DRILL COMMISSIONER'S TOUR OF INDIAN POINT FACILITY, ACCIDENT MITIGATION FEATURES FOR IP AND ZION LICENSEE PLAN FOR COPING WITH STRIKES, CONFIRMATORY ORDERS - REVIEW AND VERIFICATION OF INTERIM MEASURES - 90 DAY REPORTS, IAL ACTIONS, TURBINE ROTOR DISC INSPECTION. NO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED.

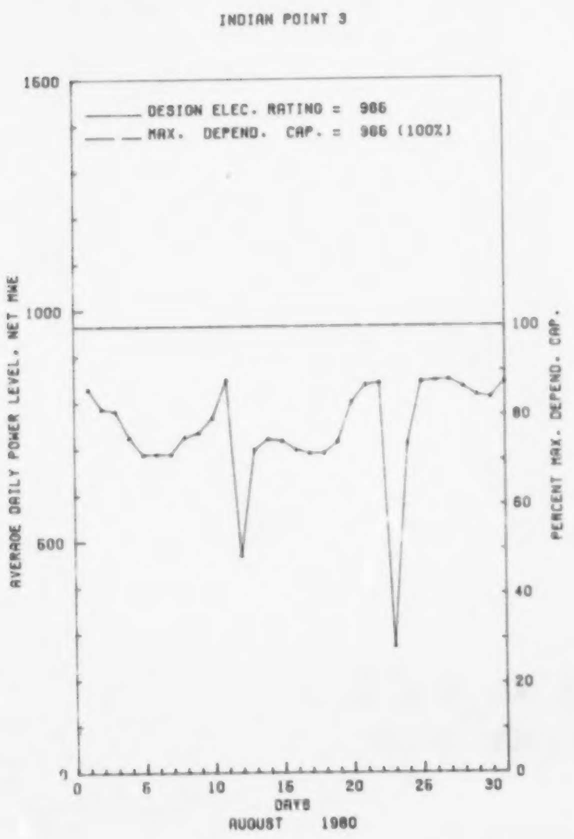
50-286/80-09 - JUN 24-26: SPECIAL, ANNOUNCED INSPECTION BY ONE REGION-BASED INSPECTOR (21 HRS) OF AN EVENT ON JUNE 24, 1980 DURING WHICH PERSONNEL PERFORMING WORK ON INCORE FLUX MONITOR SYSTEM COMPONENTS RECEIVED SIGNIFICANT PERSONNEL EXPOSURE AND CONTAMINATION. AREAS INSPECTED INCLUDED: EVENT DESCRIPTION, PERSONNEL CONTAMINATION AND INTAKE ESTIMATES, PERSONNEL EXPOSURE AND MONITORING, MAINTENANCE AND RADIATION PROTECTION PROCEDURES, NOTIFICATIONS, AND TRAINING. THE INSPECTOR OBSERVED LICENSEE REPRESENTATIVES PERFORMING RADIATION DOSE RATE MEASUREMENTS FOR USE IN PERSONNEL EXPOSURE DETERMINATION. SIX ITEMS OF NONCOMPLIANCE WERE IDENTIFIED: (1) FAILURE TO MAINTAIN RADIATION EXPOSURES WITH 10 CFR 20.101 LIMITS; (2) FAILURE TO PROVIDE PERSONNEL MONITORING; (3) FAILURE TO ESTABLISH PROCEDURES FOR INCORE FLUX DETECTOR REPAIR/REPLACEMENT; (4) FAILURE TO ADHERE TO CONTAINMENT ENTRY PROCEDURE; (5) FAILURE TO NOTIFY THE NUCLEAR REGULATORY COMMISSION; (6) FAILURE TO PREPARE PROCEDURES CONSISTENT WITH 10 CFR 20.

REVIEWED BY OMPA LOEB: T.C. Cintula **DATE:** 10·6·80

REVIEWED BY NRR DL: **DATE:** 10-7-80

INDIAN POINT 3

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-286
2. Utility Contact: C. Connell (914) 739-8200 x242
3. Licensed Thermal Power (MWt): 3025
4. Nameplate Rating (Gross MWe): 1126 x 0.9 = 1013
5. Design Electrical Rating (Net MWe): 965
6. Maximum Dependable Capacity (Gross MWe): 1000
7. Maximum Dependable Capacity (Net MWe): 965
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): 917
10. Reasons for Restrictions, If Any: REMOVED TURBINE BLADES

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	5,855.0	35,112.1
12. Hours Reactor Was Critical	738.6	4,187.6	25,772.5
13. Reactor Reserve Shutdown Hours	0	0	0
14. Hours Generator On-Line	733.1	3,727.9	24,760.7
15. Unit Reserve Shutdown Hours	0	0	0
16. Gross Thermal Energy (MWH)	1,999,791	8,895,499	64,999,956
17. Gross Electrical Energy (MWH)	551,280	2,479,000	20,844,101
18. Net Electrical Energy (MWH)	526,916	2,355,181	19,998,618
19. Unit Service Factor	98.5	63.7	70.5
20. Unit Availability Factor	98.5	63.7	70.5
21. Unit Capacity Factor (MDC Net)	73.4	41.7	59.0
22. Unit Capacity Factor (DER Net)	73.4	41.7	59.0
23. Unit Forced Outage Rate	1.5	21.9	7.3

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): TURBINE OUTAGE - OCTOBER 1980.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
26	08/12/80	F	3.2	A	3		HF	PUMPXX	UNIT TRIP DURING RAPID LOAD REDUCTION CAUSED BY LOSS OF ALL CIRCULATING WATER TO #33 CONDENSER BAY WHEN #36 CIRCULATING WATER PUMP TRIPPED WITH #35 CIRCULATING WATER PUMP PREVIOUSLY OUT OF SERVICE FOR REPAIRS. REPAIRED PUMP.
27	08/23/80	F	7.7	A	3		EB	GENERA	UNIT TRIP WHILE PLACING REPAIRED MOTOR GENERATOR SET IN SERVICE. MALFUNCTION WITH AUTO SYNCHRONIZING DEVICE. REPAIRED AUTO SYNCHRONIZING DEVICE.

SUMMARY

INDIAN POINT 3 OPERATED FOR THE MONTH OF AUGUST WITH TWO OUTAGES CAUSED BY EQUIPMENT FAILURE.

LEGEND

[1] Type:
F-Forced
S-Scheduled

[2] Reason:
A-Equipment Failure
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error
H-Other

[3] Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Continuations
5-Load Reduction
9-Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

KEWAUNEE

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Wisconsin/Kewaunee 27 miles E of Green Bay
2.	Electric Reliability Council	Mid America Interpool Network
3.	Type of Reactor	PWR
4.	Power Level (MWT)	1650
5.	Design Electrical Rating (MWE Net)	535
6.	Date of Initial Criticality	March 7, 1974
7.	Date of Commercial Operation	June 1974
8.	Condenser Cooling Method	Once through condenser and return to Lake Michigan
9.	Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10.	Licensee	Wisconsin Public Service Corp.
11.	Corporate Address	P. O. Box 1200 Green Bay, Wisconsin 54305
12.	Corporate Contact	E. R. Mathews Power Supply & Eng.
13.	Architect/Engineer	Fluor Power Services Corp.
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Pioneer Services & Eng. Co.

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	R. F. Warnick
19.	IE Resident Inspector	R. L. Nelson
20.	Licensing Project Manager	R. B. Licciardo X28362
21.	Docket Number	50-305
22.	License Number/Date of Issue	DPR-43 issued 12/21/73
23.	Public Document Room	Kewaunee Public Library 822 Juneau Street Kewaunee, Wisconsin 54216

INSPECTION STATUS

ENFORCEMENT STATUS

TECHNICAL SPECIFICATION 6.8.1 STATES, THAT "WRITTEN PROCEDURES AND ADMINISTRATIVE POLICES SHALL BE ESTABLISHED, IMPLEMENTED AND MAINTAINED THAT MEET THE REQUIREMENTS AND RECOMMENDATIONS OF SECTION 5.1 AND 5.3 OF ANSI 18.7-1972." SECTION 5.1 6.1 OF ANSI 18.7-1972, STATES THAT "MAINTENANCE THAT CAN AFFECT THE PERFORMANCE OF SAFETY-RELATED EQUIPMENT SHALL BE PROPERLY PREPLANNED AND PERFORMED IN ACCORDANCE WITH WRITTEN PROCEDURES, DOCUMENTED INSTRUCTIONS OR DRAWINGS APPROPRIATE TO THE CIRCUMSTANCES..." KEWAUNEE NUCLEAR POWER PLANT ACD 5.2 REVISION C, DATED FEBRUARY 1, 1978 STATES UNDER 5.1 AND 5.1.2 "REQUIRED PROCEDURES." "CORRECTIVE MAINTENANCE PROCEDURES OR CORRECTIVE/REPAIR MAINTENANCE PROCEDURE SHALL BE PREPARED FOR COMPONENTS IN SAFETY-RELATED SYSTEMS THAT NEED REPAIR OR REPLACEMENT, WHEN THE TYPE OF MAINTENANCE IS OUTSIDE THE SCOPE OF THE VENDOR'S MANUAL, PMP OR A GMP." CONTRARY TO THE ABOVE, EXTENSIVE REPAIRS AND A MODIFICATION WERE MADE TO THE 1B RESIDUAL HEAT REMOVAL PUMP DURING THE PERIOD OF MAY 25, 1980 THROUGH JUNE 1, 1980. THIS WORK WAS PERFORMED WITHOUT AN APPROVED PROCEDURE.
(8015 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

The plant operated at power during most of the month.

LAST IE SITE INSPECTION DATE: July 1-31, 1980

INSPECTION REPORT NO: 80-16

SUMMARY

INSPECTION SUMMARY

Inspection on July 1-3, 7-11, 14-18, 21-25, 28-31, (80-16): Operational Safety Verification, Monthly Maintenance Observation, Monthly Surveillance Observation, Licensee Event Reports, Followup on IE Bulletins, Followup on IE Circulars, Plant Trips, Organization and Administration, and Followup on Headquarters Requests, and Licensee Training Staff. The inspection involved 174 inspector-hours onsite by three NRC inspectors including 32 inspector-hours onsite during off-shifts. Of the 10 areas inspected, no items of noncompliance were identified.

REPORTS RECEIVED FROM LICENSEE

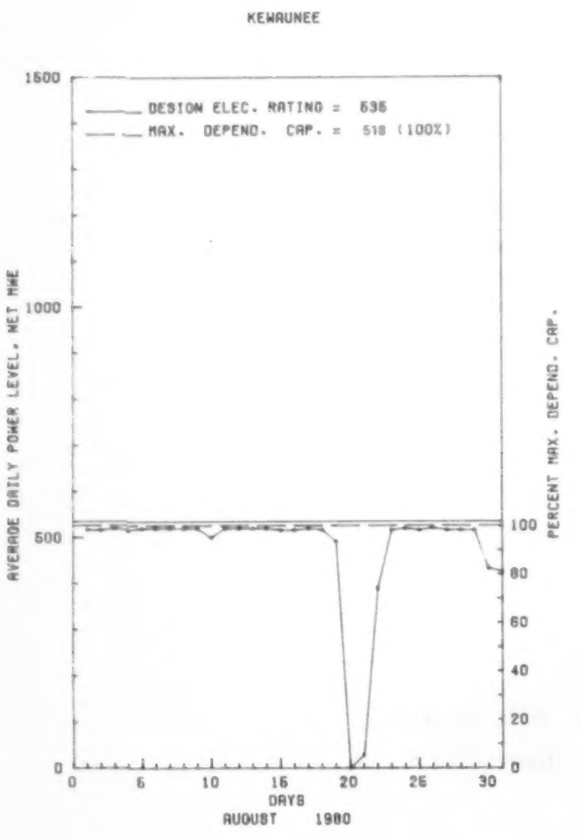
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-29/ 01T-0	07/08/80	07/29/80	Charcoal filter samples exhibited inadequate iodine removal efficiency.
80-30/ 03L-0	07/10/80	08/08/80	Instrument drift on the overpower trip bistable.

REVIEWED BY OMPA LOEB: T.C. Cintula DATE: 10·6·80

REVIEWED BY NRR DL: R. Licciardo DATE: 10-8-80

KEWAUNEE

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-305
2. Utility Contact: G.RUITER (414) 388-2560 X207
3. Licensed Thermal Power (MWt): 1650
4. Nameplate Rating (Gross MWe): 622 x 0.9 = 560
5. Design Electrical Rating (Net MWe): 535
6. Maximum Dependable Capacity (Gross MWe): 545
7. Maximum Dependable Capacity (Net MWe): 518
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
MDC CHANGED-SUMMER RATING
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

ITEM 21 CALCULATED BY USING A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	5,855.0	56,456.0
12. Hours Reactor Was Critical	705.5	4,520.7	45,523.2
13. Reactor Reserve Shutdown Hours	.0	152.7	2,330.5
14. Hours Generator On-Line	701.7	4,427.6	44,423.6
15. Unit Reserve Shutdown Hours	.0	.0	10.0
16. Gross Thermal Energy (MWH)	1,125,341	7,036,247	68,187,370
17. Gross Electrical Energy (MWH)	370,800	2,328,800	22,512,600
18. Net Electrical Energy (MWH)	353,058	2,219,725	21,418,803
19. Unit Service Factor	94.3	75.6	81.6
20. Unit Availability Factor	94.3	75.6	81.6
*21. Unit Capacity Factor (MDC Net)	91.6	73.2	75.0
22. Unit Capacity Factor (DER Net)	88.7	70.9	73.5
23. Unit Forced Outage Rate	5.7	7.0	5.5

24. Shutdowns Scheduled Over Next 6 Months (Type,Date,and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
7A	08/19/80	F	13.0	A	3		IA	INSTRU	A LIGHTNING STRIKE CAUSED THE FAILURE OF TWO INSTRUMENT BUS INVERTERS WHICH RESULTED IN A REACTOR/TURBINE TRIP AND INITIATION OF SAFETY INJECTION.
7B	08/20/80	F	29.3	A	9		EB	ELECON	A BUS FAULT ON THE LINE FROM THE RESERVE AUXILIARY TRANSFORMER TO BUSES 1 AND 2 EXTENDED THIS OUTAGE.

SUMMARY

THE UNIT OPERATED AT NEARLY FULL POWER FOR THE MAJORITY OF THE MONTH, WITH 2 OUTAGES RELATED TO A LIGHTNING STRIKE ON AUGUST 19, 1980.

LEGEND

[1] Type:
F-Forced
S-Scheduled

[2] Reason:
A-Equipment Failure
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error
H-Other

[3] Method:
1-Manual
2-Manual Scram.
3-Automatic Scram.
4-Continuations
5-Load Reduction
9-Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

LACROSSE

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Wisconsin/Vernon 19 miles S of LaCrosse
2.	Electric Reliability Council	Mid-Continent Area Reliability Coordination Agreement
3.	Type of Reactor	BWR
4.	Power Level (MWT)	165
5.	Design Electrical Rating (MWE Net)	50
6.	Date of Initial Criticality	July 11, 1967
7.	Date of Commercial Operation	November 1, 1969
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Mississippi River

Utility & Contractor Information

10.	Licensee	Dairyland Power Cooperative
11.	Corporate Address	2615 East Avenue South La Crosse, Wisconsin
12.	Corporate Contact	F. Linder
13.	Architect/Engineer	Sargent & Lundy
14.	Nuclear Steam Supply System	Allis-Chalmers
15.	Constructor	Maxson Construction

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. F. HEISHMAN
18.	IE Section Chief	D. C. BOYD
19.	IE Resident Inspector	W. Forney
20.	Licensing Project Manager	J. J. Shea
21.	Docket Number	50-409
22.	License Number/Date of Issue	DPR-45 issued 8/28/73
23.	Public Document Room	LaCrosse Public Library 800 Main Street LaCrosse, Wisconsin 54601

INSPECTION STATUS

ENFORCEMENT STATUS

LACBWR TECHNICAL SPECIFICATIONS, PARAGRAPH 3.8.1 REQUIRES "WRITTEN PROCEDURES AND ADMINISTRATIVE POLICIES SHALL BE ESTABLISHED, IMPLEMENTED..." AND LACBWR ADMINISTRATIVE CONTROL PROCEDURES (ACP-04.1) "DESIGN AND FACILITY CHANGE CONTROL" PARAGRAPH 4.18 REQUIRES, "ANY CHANGE, ADDENDA, OR ADDITIONAL PROCEDURE DEVELOPED TO IMPLEMENT OR COMPLETE AN APPROVED FACILITY CHANGE AND WHICH DID NOT ACCOMPANY THE INITIAL FACILITY CHANGE, MUST PROCEED THROUGH THE SAME REVIEW PROCESS AS THE ORIGINAL FACILITY CHANGE. THE REVIEW AND APPROVAL SHALL BE DOCUMENTED ON FACILITY CHANGE FORM 2..." CONTRARY TO THE ABOVE, ON JUNE 26, 1980, WHILE PERFORMING CONTAINMENT BUILDING ISOLATION TEST 4.1 OF VOLUME XI OF LACBWR OPERATING MANUAL, IT WAS DISCOVERED THAT THE SHUTDOWN CONDENSER CONDENSATE LEG DRAIN VALVE (S/DCCLDV) DID NOT CLOSE ON CONTAINMENT BUILDING HIGH PRESSURE, OR REACTOR LOW WATER LEVEL AS REQUIRED. SUBSEQUENTLY, LACBWR PERSONNEL PERFORMED SYSTEM WIRING MODIFICATIONS TO THE CONTROL CIRCUITRY FOR THE S/DCCLDV WITHOUT FOLLOWING THE PROCEDURES AS DELINEATED IN ACP-04.1.
(8006 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

FACILITY ITEMS

Refueling shutdown is presently scheduled for the last week in November (80-06)

MANAGERIAL ITEMS

All open operator positions will be filled in August. The position of Health and Safety Engineer has been filled. A new position, Technical Support Engineer has been established. The positions of Mechanical Engineer and Nuclear Engineer are still open. (80-06)

PLANT STATUS

Plant in cold shutdown status to replace several leaking CRD seals. Tentative restart date is September 3, 1980.

LAST IE SITE INSPECTION DATE: July 1-31, 1980

INSPECTION REPORT NO: 80-06

INSPECTION SUMMARY

Inspection on July 1-31, (80-06): Routine, announced inspection of the licensee's operational safety; surveillance; maintenance; followup action to IE Bulletins; IE Circulars and open inspection items; organization and administration; onsite review committee and procedures. The inspection involved 403 inspector-hours onsite by four NRC inspectors including 54 inspector-hours onsite during off-shifts. Of the nine areas inspected, no items of noncompliance or deviations were found in eight areas; one item of noncompliance was found in one area (failure to follow procedures in making a facility change).

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
NONE				

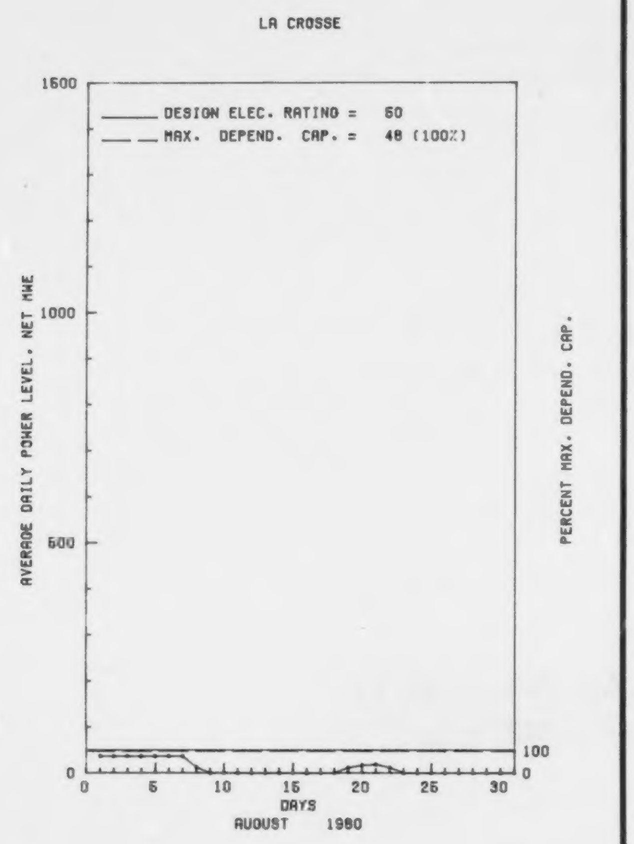
REVIEWED BY OMPA LOEB: John R Crooks **DATE:** 10/6/80

REVIEWED BY NRR DL: J J Shea **DATE:** 10/7/80

best copy available

LACROSSE

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-409
2. Utility Contact: L. S. Goodman (608) 689-2331
3. Licensed Thermal Power (MWt): 165
4. Nameplate Rating (Gross MWe): 76.8 x 0.85 =65
5. Design Electrical Rating (Net MWe): 50
6. Maximum Dependable Capacity (Gross MWe): 50
7. Maximum Dependable Capacity (Net MWe): 48
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	5,855.0	94,970.0
12. Hours Reactor Was Critical	337.0	4,702.9	62,509.0
13. Reactor Reserve Shutdown Hours	.0	.0	478.0
14. Hours Generator On-Line	264.8	4,501.8	57,506.8
15. Unit Reserve Shutdown Hours	.0	.0	79.0
16. Gross Thermal Energy (MWH)	32,467	597,267	7,882,046
17. Gross Electrical Energy (MWH)	8,695	172,871	2,372,003
18. Net Electrical Energy (MWH)	7,496	160,797	2,193,532
19. Unit Service Factor	35.6	76.9	60.6
20. Unit Availability Factor	35.6	76.9	60.6
21. Unit Capacity Factor (MDC Net)	21.0	57.2	48.1
22. Unit Capacity Factor (DER Net)	20.2	54.9	46.2
23. Unit Forced Outage Rate	64.4	14.2	7.1

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUELING OUTAGE, NOVEMBER 22, 1980 (6 WEEKS).
25. If Shut Down At End of Report Period, Estimated Date of Startup: 09/04/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-07	08/08/80	F	253.7	A	3		CJ	INSTRU	FORCED CIRCULATION PUMPS TRIPPED DUE TO LOSS OF SEAL INJECTION FLOW. LOSS OF SEAL INJECTION FLOW WAS CAUSED BY LOW LEVEL IN RESERVOIR DUE TO LEVEL CONTROLLER MALFUNCTION. SEAL INJECT RESERVOIR LEVEL CONTROLLERS AND INDICATORS WERE RECALIBRATED.
80-08	08/22/80	F	225.5	A	3		CH	INSTRU	THE SHUTDOWN RESULTED WHEN REACTOR FEED PUMP 1A TRIPPED DUE TO ELECTRICAL SHORT ON A PRINTED CIRCUIT CONTROL CARD CAUSED BY WATER ENTERING CONTROL ENCLOSURE. DECONTAMINATION OF OVERHEAD FLOOR CAUSED WATER TO DRIP THROUGH SMALL UNSEALED CONDUIT CHASE. THE CONDUIT CHASE WAS SEALED. WHILE THE PLANT WAS SHUTDOWN, MAINTENANCE WAS ALSO PERFORMED ON THE SEAL INJECTION SYSTEM AND THE MECHANICAL SEALS IN CONTROL ROD DRIVE MECHANISMS NOS. 5 AND 21.

SUMMARY

LA CROSSE OPERATED WITH TWO OUTAGES - ON 08/03/80 DUE TO LOSS OF INJECTION FLOW, AND ON THE 22ND, DUE TO AN ELECTRICAL SHORT CAUSED BY WATER ENTERING THE CONTROL ENCLOSURE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

MAINE YANKEE

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Maine/Lincoln 10 miles N of Bath (near Wiscasset)
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2630
5.	Design Electrical Rating (MWE Net)	825
6.	Date of Initial Criticality	October 23, 1972
7.	Date of Commercial Operation	December 28, 1972
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Back River (Atlantic)

Utility & Contractor Information

10.	Licensee	Maine Yankee Atomic Power Co.
11.	Corporate Address	20 Turnpike Road Westboro, Massachusetts 01581
12.	Corporate Contact	D. W. Edwards, Assistant to Vice President
13.	Architect/Engineer	Stone & Webster
14.	Nuclear Steam Supply System	Combustion Engineering
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	IE Section Chief	T. T. Martin
19.	IE Resident Inspector	W. J. LAZARUS (X5258)
20.	Licensing Project Manager	G. Requa (X28478)
21.	Docket Number	50-309
22.	License Number/Date of Issue	DPR-36, 9/15/72
23.	Public Document Room	Wiscasset Public Library High Street Wiscasset, Maine 04578

INSPECTION STATUS

ENFORCEMFNT STATUS

PA ACCESS CONTROLLED AREA BARRIER (DOOR) FOUND UNLOCKED.
(8008 INFRACTION)

EXPLOSIVES DETECTOR COULD NOT DETECT TEST SOURCE.
(8008 INFRACTION)

VA LOCKING DOOR ASSEMBLY NOT MANIPULATION RESISTANT.
(8008 INFRACTION)

THE LICENSEE FAILED TO TEST A "B" DIESEL GENERATOR WITHIN TWO HOURS OF TAKING "A" SCC PUMP OUT OF SERVICE IAW T.S. 3.6.2.B.
(8009 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEM AND COMPONENT PROBLEMS:

THE "A" MAIN FEED PUMP FAILED ON AUGUST 2, RESTRICTING THE PLANT TO 65% POWER UNTIL REPAIRS WERE COMPLETED ON AUGUST 30.

PLANT STATUS:

POWER OPERATION WAS RESTRICTED TO 65% FROM AUGUST 2 UNTIL REPAIRS WERE COMPLETED TO THE "A" MAIN FEED PUMP ON AUGUST 30. FULL POWER WAS REACHED ON AUGUST 31.

LAST IE SITE INSPECTION DATE: 8/25-29/80

INSPECTION REPORT NO: 50-309/80-12

SUMMARY

INSPECTION SUMMARY

50-309/80-09 - JUN 2 - JUL 2: ROUTINE, ONSITE, REGULAR AND BACKSHIFT INSPECTION BY TWO RESIDENT INSPECTORS (107 HRS). AREAS INSPECTED INCLUDED THE CONTROL ROOM, TURBINE BUILDING, PRIMARY AUXILIARY BUILDING, SPENT FUEL BUILDING, SPRAY BUILDING AND AUXILIARY FEED PUMP AREAS. ACTIVITIES/RECORDS INSPECTED INCLUDED RADIATION PROTECTION, PHYSICAL SECURITY, PLANT OPERATIONS, MAINTENANCE, AND SURVEILLANCE TESTING. ONE ITEM OF NONCOMPLIANCE WAS IDENTIFIED: FAILURE TO TEST REDUNDANT SAFEGUARDS EQUIPMENT WHEN REQUIRED.

50-309/80-10 - JUL 7 - AUG 6: ROUTINE INSPECTION OF PLANT OPERATIONS BY TWO RESIDENT INSPECTORS (84 HRS), INCLUDING PLANT TOURS; REVIEW OF SHIFT LOGS AND RECORDS; REVIEW OF LICENSEE ACTION ON SELECTED PREVIOUS INSPECTION FINDINGS AND SELECTED IE BULLETINS AND CIRCULARS; OBSERVATION OF PHYSICAL SECURITY; FOLLOWUP ON NRR LETTER 3/28/80 (TI 2515/36); FOLLOWUP ON LICENSEE IMPLEMENTATION OF SMALL BREAK LOSS OF COOLANT ACCIDENT CRITERIA (TI 2515/32); AND REVIEW OF EVENTS OCCURRING DURING THE INSPECTION. NO ITEMS OF NON-COMPLIANCE WERE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

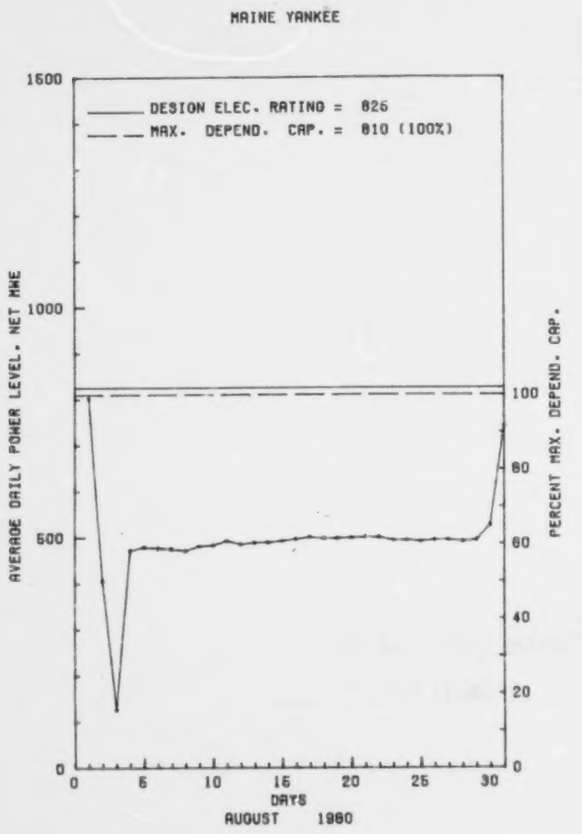
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-012/ 03L	05/06/80	06/03/80	LSI-M-31 FAILED TO OPERATE DURING SURVEILLANCE TESTING.
80-013/ 03L	05/07/80	06/04/80	IMPROPER LINEUP IN HPSI COOLING AND LUBE OIL PUMPS
80-015/ 01T	06/25/80	07/09/80	ERROR IN CYCLE 5 SAFETY ANALYSIS.
80-016/ 03L	06/02/80	07/02/80	NO. 1 STEAM GENERATOR NON-RETURN VALVE FAILURE.

REVIEWED BY OMPA LOEB: MA Beebe **DATE:** 10/06/80

REVIEWED BY NRR DL: G. Requa **DATE:** 10/08/80

MAINE YANKEE

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-309
2. Utility Contact: F.C. BEERS (617) 366-9011
3. Licensed Thermal Power (MWt): 2630
4. Nameplate Rating (Gross MWe): 900 x 0.9 = 810
5. Design Electrical Rating (Net MWe): 825
6. Maximum Dependable Capacity (Gross MWe): 850
7. Maximum Dependable Capacity (Net MWe): 810

CAPACITY FACTORS CALCULATED WITH A WEIGHTED AVERAGE

8. If Cnanges Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	5,855.0	68,483.6
12. Hours Reactor Was Critical	733.1	4,073.3	55,283.2
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	718.6	3,879.1	53,396.8
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,257,392	8,294,490	113,536,573
17. Gross Electrical Energy (MWH)	390,760	2,713,660	37,286,840
18. Net Electrical Energy (MWH)	367,952	2,579,275	35,406,461
19. Unit Service Factor	96.6	66.3	78.0
20. Unit Availability Factor	96.6	66.3	78.0
*21. Unit Capacity Factor (MDC Net)	61.1	54.4	67.0
22. Unit Capacity Factor (DER Net)	59.9	53.4	64.7
23. Unit Forced Outage Rate	3.4	9.1	7.1

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
-	08/02/80	F	25.4	A	3		HH	PUMPXX	MAJOR FAILURE OF THE P-2A FEEDWATER PUMP. UNABLE TO DETERMINE EXACT CAUSE OF FAILURE. NEW PUMP AND ROTATING ASSEMBLY INSTALLED. PLANT RECOVERED FROM THE TRIP AND RETURNED TO 60-65% FOR ALMOST THE REMAINDER OF THE MONTH. P-2A WAS BACK IN SERVICE ON 08/30/80 AND THE PLANT WAS BACK AT FULL LOAD ON 08/31/80.

SUMMARY

MAINE YANKEE OPERATED AT REDUCED POWER FOR THE MONTH, WITH ONE OUTAGE ON AUGUST 2ND FOR FEEDWATER PUMP FAILURE.

LEGEND

[1] Type:
F- Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram.
3--Automatic Scram.
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

MILLSTONE 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1. Location (state/county) (nearest population center): Connecticut/New London, 5 miles SW of New London
2. Electric Reliability Council: Northeast Power Coordinating Council
3. Type of Reactor: BWR
4. Power Level (MWT): 2011
5. Design Electrical Rating (MWE Net): 660
6. Date of Initial Criticality: October 26, 1970
7. Date of Commercial Operation: March 1971
8. Condenser Cooling Method: Once Through
9. Condenser Cooling Water Source: Long Island Sound

Utility & Contractor Information

10. Licensee: Northeast Nuclear Energy Company
11. Corporate Address: P. O. Box 270, Hartford, Connecticut
12. Corporate Contact: D. Switzer, President
13. Architect/Engineer: Ebasco
14. Nuclear Steam Supply System: GE
15. Constructor: Ebasco

Regulatory Information

16. IE Region responsible: I
17. IE Branch Chief: E. Brunner
18. IE Section Chief: F. KEIMIG
19. IE Resident Inspector: J. SHEDLOSKY (203-447-3179) FTS: 642-7287
20. Licensing Project Manager: J. J. Shea
21. Docket Number: 50-245
22. License Number/Date of Issue: DPR-21, 10/7/70
23. Public Document Room: Waterford Public Library, Rope Ferry Road, Route 156, Waterford, Connecticut 06

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO T.S. 6.8.1, ON APRIL 23, 1980, A PLANT EMPLOYEE WAS OBSERVED WORKING IN THE UNIT 1 REACTOR BUILDING, A RADIATION AREA WITH DOSE RATES GREATER THAN 0.5 MREM/HOUR, WITHOUT PERSONAL FILM BADGE/THERMOLUMINESCENT DOSIMETER.
(8006 INFRACTION)

T.S. 6.10.1D REQUIRES RECORDS OF SURVEILLANCE ACTIVITIES INSPECTIONS AND CALIBRATIONS BE RETAINED FOR 5 YEARS. CONTRARY TO THE ABOVE RESULTS WERE NOT MAINTAINED FOR SURVEILLANCE, SP 680 M, ANNUAL FIRE PROTECTION SYSTEM PUMP FLOW RATE TEST PERFORMED IN 1979.
(8008 DEFICIENCY)

T.S. 4.13 REQUIRES INSPECTION AND TESTING WITH APPROPRIATE EXCEPTION AND ALTERNATIVE INSPECTIONS AND TESTING AS PER INSERVICE INSPECTION PROGRAM. CONTRARY TO ABOVE (1) VALVE 1-RC-6, RBCCW HEADER INLET CHECK WAS NOT EXERCISED AS REQUIRED (2) VALVE 301-138, CRD-RETURN ISOLATION WAS NOT VERIFIED OPEN AS REQUIRED AND (3) VALVE 1-IC-12 MAKEUP ISO. COND. STOP WAS NOT VERIFIED IN THE "LOCKED OPEN" POSITION. 1-IC-12 WAS OPEN BUT NOT "LOCKED OPEN".
(8008 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS:

THE UNIT IS OPERATING AT FULL POWER.

LAST IE SITE INSPECTION DATE: 8/11-22/80

INSPECTION REPORT NO: 50-245/80-14

SUMMARY

INSPECTION SUMMARY

50-245/80-14 - AUG 5: AN ENFORCEMENT MEETING WAS HELD AT THE REQUEST OF NRC, REGION I, TO DISCUSS THE PHYSICAL SECURITY PROGRAM AND TO AGREE ON THE CONDITIONS ISSUED IN AN IMMEDIATE ACTION LETTER (IAL 80-28).

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-011/01T	08/03/80	08/15/80	AIR EJECTOR OFFGAS RADIATION MONITORS FOUND ISOLATED DUE TO MIS-POSITIONED MANUAL VALVE.

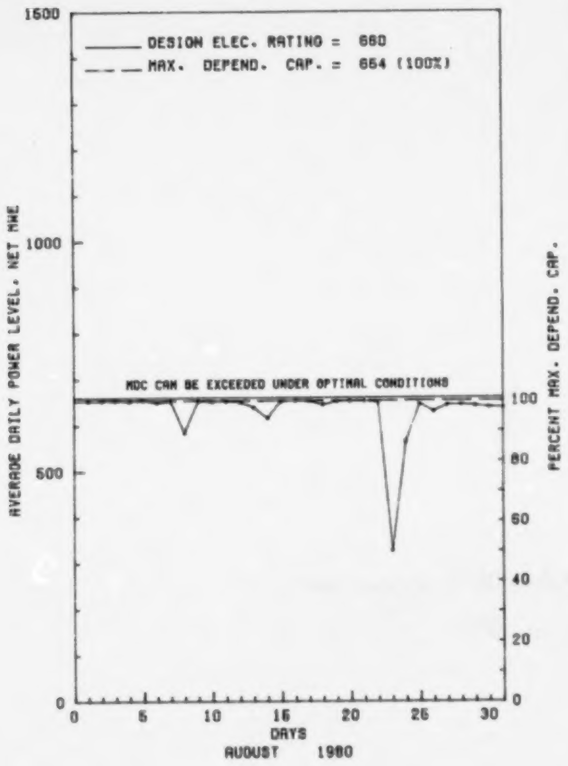
REVIEWED BY OMPA LOEB: John G. Crooks **DATE:** 10/6/80

REVIEWED BY NRR DL: J J Shea **DATE:** 10/6/80

MILLSTONE 1

AVERAGE DAILY POWER LEVEL (MWe)

MILLSTONE 1



OPERATING STATUS

1. Docket Number: 50-245
2. Utility Contact: G. HARRAN (203) 447-1792 X655
3. Licensed Thermal Power (MWt): 2011
4. Nameplate Rating (Gross MWe): 735 x 0.9 = 662
5. Design Electrical Rating (Net MWe): 660
6. Maximum Dependable Capacity (Gross MWe): 684
7. Maximum Dependable Capacity (Net MWe): 654
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	5,855.0	85,535.0
12. Hours Reactor Was Critical	744.0	5,266.0	65,651.0
13. Reactor Reserve Shutdown Hours	.0	592.3	1,527.3
14. Hours Generator On-Line	744.0	5,234.0	63,329.9
15. Unit Reserve Shutdown Hours	.0	26.5	26.5
16. Gross Thermal Energy (MWH)	1,438,111	8,990,457	113,0[illegible]
17. Gross Electrical Energy (MWH)	492,600	3,059,947	38,291,343
18. Net Electrical Energy (MWH)	474,635	2,922,206	36,561,988
19. Unit Service Factor	100.0	89.4	74.0
20. Unit Availability Factor	100.0	89.8	74.1
21. Unit Capacity Factor (MDC Net)	97.5	76.3	65.4
22. Unit Capacity Factor (DER Net)	96.7	75.6	64.8
23. Unit Forced Outage Rate	.0	.3	15.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
REFUELING & MAINTENANCE - 10/04/80 - 9-11 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	NONE								

SUMMARY

MILLSTONE 1 OPERATED AT NEAR FULL POWER FOR THE MONTH, WITH EXCEPTION OF A REDUCTION TO REPAIR LEAKING MAIN CONDENSER TUBES ON THE 23RD.

LEGEND

[1] Type:
F-Forced
S-Scheduled

[2] Reason:
A-Equipment Failure
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error
H-Other

[3] Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Continuations
5-Load Reduction
9-Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

MILLSTONE 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Connecticut/New London 5 miles SW of New London
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2700
5.	Design Electrical Rating (MWE Net)	870
6.	Date of Initial Criticality	October 17, 1975
7.	Date of Commercial Operation	December 26, 1975
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Long Island Sound

Utility & Contractor Information

10.	Licensee	Northeast Nuclear Energy Company
11.	Corporate Address	P. O. Box 270 Hartford, Connecticut
12.	Corporate Contact	D. Switzer, President
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Combustion Engineering
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	IE Section Chief	E. McCabe
19.	IE Resident Inspector	J. T. SHEDLOSKY (203-447-3179) FTS: 642-7287 R. Zimmerman
20.	Licensing Project Manager	E. L. Conner, Jr.
21.	Docket Number	50-336
22.	License Number/Date of Issue	DPR-65, 8/1/75
23.	Public Document Room	Waterford Public Library Rope Ferry Road Route 156 Waterford, Connecticut 06385

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS:

THE THIRD REFUELING OUTAGE COMMENCED ON AUGUST 16, 1980 AND IS EXPECTED TO LAST APPROXIMATELY NINE WEEKS.

LAST IE SITE INSPECTION DATE: 8/27-29/80

INSPECTION REPORT NO: 50-336/80-13

SUMMARY

INSPECTION SUMMARY

50-336/80-10 - JUL 28-30: ROUTINE ANNOUNCED INSPECTION BY ONE REGION-BASED INSPECTOR (16 HRS) OF QUALIFICATION DATA FOR THE ELECTRICAL EQUIPMENT INSIDE CONTAINMENT ASSOCIATED WITH THE REACTOR COOLANT SYSTEM, AND SAFETY INJECTION AND CONTAINMENT SPRAY SYSTEM AS DEFINED IN THE CLASS IE SUBMITTAL OF JULY 15, 1980. NO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED.

50-336/80-15 - AUG 5: AN ENFORCEMENT MEETING WAS HELD AT THE REQUEST OF NRC, REGION I, TO DISCUSS THE PHYSICAL SECURITY PROGRAM AND TO AGREE ON THE CONDITIONS ISSUED IN AN IMMEDIATE ACTION LETTER (IAL 80-28).

REPORTS RECEIVED FROM LICENSEE

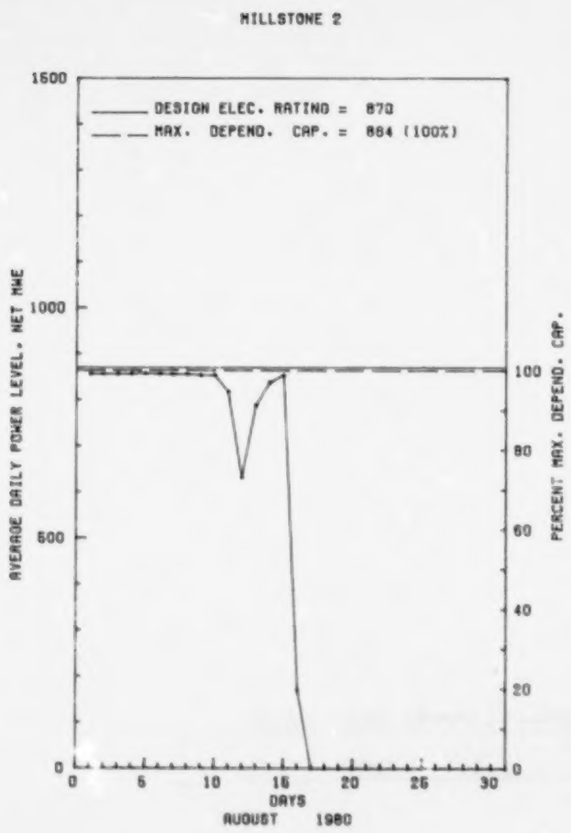
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-024/ 03L	07/03/80	08/04/80	"A" HPSI PUMP SEIZED FOLLOWING OPERATION WITH MINIMUM FLOW VALVE SHUT.
80-025/ 03L	07/12/80	08/07/80	TURBINE DRIVEN AUXILIARY FEED PUMP TRIPPED BELOW NORMAL SETPOINT DUE TO WORN TAPPET NUT.

REVIEWED BY OMPA LOEB: MK Beebe **DATE:** 10/06/80

REVIEWED BY NRR DL: C Nelson **DATE:** 10/07/80

MILLSTONE 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-336
2. Utility Contact: G. H. Howlett (203) 447-1791 x364
3. Licensed Thermal Power (MWt): 2700
4. Nameplate Rating (Gross MWe): 1011 x 0.9 = 910
5. Design Electrical Rating (Net MWe): 870
6. Maximum Dependable Capacity (Gross MWe): 895
7. Maximum Dependable Capacity (Net MWe): 864
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

CAPACITY FACTORS CALCULATED WITH A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	5,855.0	41,063.0
12. Hours Reactor Was Critical	368.5	4,276.7	29,355.4
13. Reactor Reserve Shutdown Hours	.0	.0	2,072.4
14. Hours Generator On-Line	368.0	4,183.6	28,003.8
15. Unit Reserve Shutdown Hours	.0	.0	335.4
16. Gross Thermal Energy (MWH)	958,023	11,026,274	68,718,904
17. Gross Electrical Energy (MWH)	315,565	3,616,072	22,198,717
18. Net Electrical Energy (MWH)	302,334	3,471,253	21,251,511
19. Unit Service Factor	49.5	71.5	68.2
20. Unit Availability Factor	49.5	71.5	69.0
*21. Unit Capacity Factor (MDC Net)	47.0	68.6	62.8
22. Unit Capacity Factor (DER Net)	46.7	68.1	61.8
23. Unit Forced Outage Rate	.0	23.6	21.8

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 10/18/80

UNIT SHUTDOWN/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
12	08/11/80	F	.0	H	5	80-28	RB	CRDRVE	DROPPED C.E.A. #41 FORCED A POWER REDUCTION (70%) PER SPECS. UPON RETRIEVAL OF C.E.A. #41 A SECOND C.E.A. #65 DROPPED. THE APPARENT CAUSE WAS DUE TO A FAULTY JUMPER CABLE. THE ROD WAS REALIGNED, WITH SUBSEQUENT NORMAL POWER OPERATIONS.
13	08/16/80	S	376.0	C	1				COMMENCED CYCLE 4 REFUEL OUTAGE.

SUMMARY

MILLSTONE 2 OPERATED AT OR NEAR 100% THERMAL POWER EXCEPT FOR DOWN POWER ON THE 11TH FOR DROPPED C.E.A.'S AND SHUTDOWN ON THE 16TH FOR CYCLE 4 REFUELING.

LEGEND

[1] Type:
F–Forced
S–Scheduled

[2] Reason:
A–Equipment Failure
B–Maintenance or Test
C–Refueling
D–Regulatory Restriction
E–Operator Training & License Examination
F–Administrative
G–Operational Error
H–Other

[3] Method:
1–Manual
2–Manual Scram
3–Automatic Scram
4–Continuations
5–Load Reductions
9–Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

MONTICELLO

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Minnesota/Wright 30 miles NW of Minneapolis
2.	Electric Reliability Council	Mid-Continent Area Reliability Coordination Agreement
3.	Type of Reactor	BWR
4.	Power Level (MWT)	1670
5.	Design Electrical Rating (MWE Net)	545
6.	Date of Initial Criticality	December 10, 1970
7.	Date of Commercial Operation	June 30, 1971
8.	Condenser Cooling Method	Once Through with Cooling Tower
9.	Condenser Cooling Water Source	Mississippi River

Utility & Contractor Information

10.	Licensee	Northern States Power Co.
11.	Corporate Address	414 Nicollet Mall - 8th Floor Minneapolis, Minnesota 55401
12.	Corporate Contact	G. H. Neils (Oper.) L. O. Mayer (Lic.)
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	GE
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. F. HEISHMAN
18.	IE Section Chief	W. S. Little
19.	IE Resident Inspector	C. Brown
20.	Licensing Project Manager	K. Eccleston
21.	Docket Number	50-263
22.	License Number/Date of Issue	DPR-22 issued 9/8/70
23.	Public Document Room	Environmental Conservation Library Minneapolis Public Library 300 Nicollet Mall Minneapolis, Minnesota 55401

INSPECTION STATUS

ENFORCEMENT STATUS

10 CFR 20.203(B) REQUIRES THAT EACH RADIATION AREA SHALL BE CONSPICUOUSLY POSTED WITH A SIGN OR SIGNS BEARING THE RADIATION CAUTION SYMBOL AND THE WORDS CAUTION OR DANGER RADIATION AREA. CONTRARY TO THE ABOVE: (A) THE OUTSIDE ENTRANCE TO THE RADIATION AREA OF THE "HOT" MACHINE SHOP WAS NOT POSTED AS A RADIATION AREA, AND (B) AREAS WITHIN THE UNPOSTED TURBINE BUILDING OUTSIDE THE HOT MACHINE SHOP POSTED AREA HAD DOSE RATES OF UP TO 12 MR/HR.
(8011 INFRACTION)

TECHNICAL SPECIFICATION 6.5.B.1.A REQUIRES THAT ANY INDIVIDUAL OR GROUP OF INDIVIDUALS PERMITTED TO ENTER A HIGH RADIATION AREA SHALL BE PROVIDED WITH A RADIATION MONITORING DEVICE WHICH CONTINUOUSLY INDICATES THE RADIATION DOSE RATE IN THE AREA. CONTRARY TO THE ABOVE, A LICENSEE EMPLOYEE ACCOMPANIED BY A VISITOR WAS OBSERVED ENTERING A HIGH RADIATION AREA (951 FOOT LEVEL, TURBINE BUILDING) WITHOUT THE REQUIRED RADIATION MONITORING DEVICE.
(8011 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

MANAGERIAL ITEMS

Significant appraisal findings were found concerning: radiation protection group staffing levels, offshift capabilities, authority, and training; control of access to the controlled area; controlled area surveillance and postings; lack of a normal ALARA program; and method of selecting qualified contract radiation protection technicians. (80-11)

PLANT STATUS

Operating routinely.

LAST IE SITE INSPECTION DATE: June 1-30, 1980

INSPECTION REPORT NO: 80-14

SUMMARY

INSPECTION SUMMARY

Inspection on June 9-20, (80-11): Special, announced inspection of radiation protection program, including organization and management, training, quality assurance, procedures, internal and external exposure controls, surveys and access controls, instrumentations, ALARA, radioactive waste, facilities and equipment, and accident response capabilities. The inspection involved 500 inspector-hours onsite by five NRC inspectors. Several significant weaknesses in the radiation protection program were identified. These weaknesses are in the areas of staffing, offshift radiation protection capabilities, contract radiation protection technician qualifications, management support, ALARA program, training, access control, and radiation and contamination area surveillance and posting. Two apparent items of noncompliance were found (inadequate high radiation area access controls; and inadequate posting of radiation areas).

Inspection during June, (80-14): Routine inspection of control room operations, review of logs; selected emergency systems; followup of licensee event reports, IE Bulletins and IE Circulars; organization and administration; onsite review committee; transportation of irradiated fuel by truck. The inspection involved 62 inspector-hours onsite by two NRC inspectors including 30 inspector-hours onsite during off-shifts. No items of noncompliance or deviations were identified in these areas.

REPORTS RECEIVED FROM LICENSEE

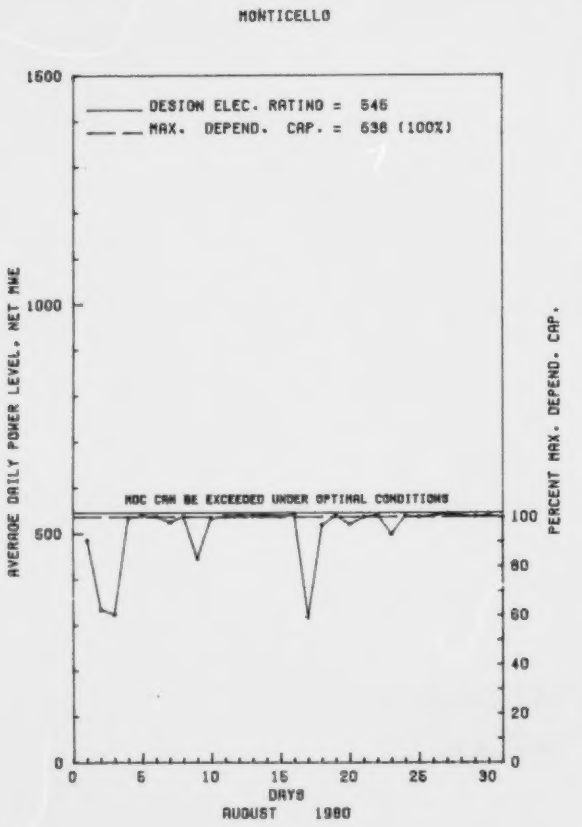
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
None.			

REVIEWED BY OMPA LOEB: John E. Crooks **DATE:** 10/6/80

REVIEWED BY NRR DL: Ken Eccleston **DATE:** 10/8/80

MONTICELLO

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-263
2. Utility Contact: A. L. Myrabo (612) 295-5151
3. Licensed Thermal Power (MWt): 1670
4. Nameplate Rating (Gross MWe): 632 x 0.9 = 569
5. Design Electrical Rating (Net MWe): 545
6. Maximum Dependable Capacity (Gross MWe): 564
7. Maximum Dependable Capacity (Net MWe): 536
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level to Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	5,855.0	80,408.0
12. Hours Reactor Was Critical	744.0	4,249.6	65,779.9
13. Reactor Reserve Shutdown Hours	.0	.0	940.7
14. Hours Generator On-Line	744.0	4,153.9	64,124.0
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,193,390	6,261,777	105,204,639
17. Gross Electrical Energy (MWH)	403,396	2,108,696	33,086,756
18. Net Electrical Energy (MWH)	383,824	2,005,002	31,637,206
19. Unit Service Factor	100.0	70.9	79.8
20. Unit Availability Factor	100.0	70.9	79.8
21. Unit Capacity Factor (MDC Net)	96.2	63.9	73.4
22. Unit Capacity Factor (DER Net)	94.7	62.8	72.2
23. Unit Forced Outage Rate	.0	11.7	6.9

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
MARCH 28, 1981 - REFUELING OUTAGE - 56 DAYS
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
11	08/02/80	F	.0	A	5		HH	HTEXCH	WATER LEAK IN THE CONDENSATE PUMP LUBE OIL COOLER. LEAK WAS REPAIRED. COOLER WILL BE REPLACED.
12	08/17/80	F	.0	A	5	80-26	CD	VALVEX	REPAIRED 4-WAY ACTUATING VALVE ON OUTBOARD MSIV AO-2-86A.

SUMMARY

MONTICELLO OPERATED AT APPROXIMATELY 90% FOR THE BULK OF THE MONTH, WITH 2 LOAD REDUCTIONS.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

NINE MILE POINT 1

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	New York/Oswego 8 miles NE of Oswego
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	1850
5.	Design Electrical Rating (MWE Net)	620
6.	Date of Initial Criticality	September 5, 1969
7.	Date of Commercial Operation	December 1969
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Lake Ontario

Utility & Contractor Information

10.	Licensee	Niagara Mohawk Power Corporation
11.	Corporate Address	300 Erie Boulevard West Syracuse, New York 13202
12.	Corporate Contact	R. Schneider, Vice President Electric Operations
13.	Architect/Engineer	Utility
14.	Nuclear Steam Supply System	GE
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Section Chief	H. B. KISTER
19.	IE Resident Inspector	W. BAUNACK (315) 342-4041 FTS: 264-3311 · Operator
20.	Licensing Project Manager	P. Polk
21.	Docket Number	50-220
22.	License Number/Date of Issue	DPR-63, 12/26/74
23.	Public Document Room	State University College of Oswego Penfield Library · Documents Oswego, NY 13126 (315)341-2323

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

```
INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS:

THE PLANT IS OPERATING AT 98% POWER DUE TO END OF
CYCLE DERATING ON FOUR RECIRCULATION PUMPS.  DURING
MOST OF THE MONTH, THE PLANT OPERATED AT 95% TO 98%
DUE TO CONDENSER DELTA T AND HOTWELL TEMPERATURE
LIMITS.  REPRESENTATIVE STACK AND OFF GAS RELEASE
RATES ARE 10 AND 27,500 MICROCURIES PER SECOND,
RESPECTIVELY.


LAST IE SITE INSPECTION DATE:  8/15/80

     INSPECTION REPORT NO:  50-220/80-07

SUMMARY

INSPECTION SUMMARY

NONE.
```

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

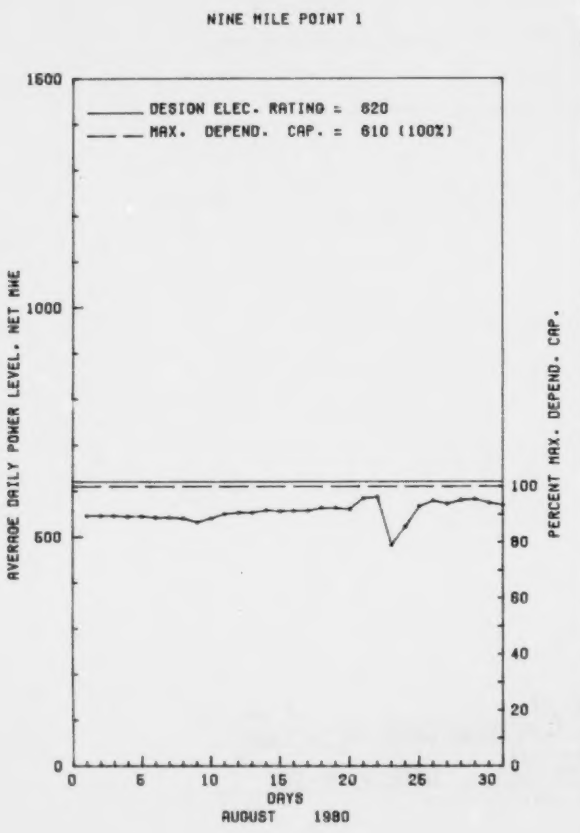
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-11	06/23/80	07/22/80	CONDENSER DELTA TEMPERATURE TECHNICAL SPECIFICATION LIMIT EXCEEDED.
80-12	04/19/80	08/01/80	TURBINE ANTICIPATORY TRIP SCRAM BYPASS SETPOINT OUTSIDE LIMITS.
80-014/03L	07/17/80	08/08/80	ELECTROMATIC RELEF VALVE FAILED TO OPEN.
80-015/04L	08/04/80	08/18/80	OFF GAS RAD MONITOR PLACED IN PURGE FOR THREE HOURS.

REVIEWED BY OMPA LOEB: John R Crooks **DATE:** 10/6/80

REVIEWED BY NRR DL: Ken Eccleston **DATE:** 10/8/80

NINE MILE POINT 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-220
2. Utility Contact: B. Hawksley (315) 343-2110 x1312
3. Licensed Thermal Power (MWt): 1850
4. Nameplate Rating (Gross MWe): 755 x 0.85 = 642
5. Design Electrical Rating (Net MWe): 620
6. Maximum Dependable Capacity (Gross MWe): 630
7. Maximum Dependable Capacity (Net MWe): 610
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	5,855.0	94,967.0
12. Hours Reactor Was Critical	744.0	5,334.7	70,687.4
13. Reactor Reserve Shutdown Hours	.0	.0	1,204.2
14. Hours Generator On-Line	744.0	5,248.7	68,059.2
15. Unit Reserve Shutdown Hours	.0	.0	20.2
16. Gross Thermal Energy (MWH)	1,327,859	9,179,119	110,920,518
17. Gross Electrical Energy (MWH)	425,775	3,049,342	36,558,852
18. Net Electrical Energy (MWH)	412,081	2,950,907	35,400,672
19. Unit Service Factor	100.0	89.6	71.7
20. Unit Availability Factor	100.0	89.6	71.7
21. Unit Capacity Factor (MDC Net)	90.8	82.6	61.1
22. Unit Capacity Factor (DER Net)	89.3	81.3	60.1
23. Unit Forced Outage Rate	.0	6.5	9.1

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	NONE								

SUMMARY

NINE MILE POINT OPERATED NORMALLY FOR THE ENTIRE MONTH, WITH NO REPORTED OUTAGES OR REDUCTIONS.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

NORTH ANNA 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Louisa County, Virginia 40 Miles NW of Richmond, Virginia
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	PWR
4.	Licenses Power Level (MWT)	2775
5.	Design Electrical Rating (MWE Net)	907
6.	Date of Initial Criticality	April 5, 1978
7.	Date of Commercial Operation	June 6, 1978
8.	Condenser Cooling Method	Once through
9.	Condenser Cooling Water Source	Lake Anna

Utility & Contractor Information

10.	Licensee	Virginia Electric & Power Co.
11.	Corporate Address	P. O. Box 26666 Richmond, Virginia 23261
12.	Corporate Contact	J. H. Ferguson Exec. V.P. - Power
13.	Architect/Engineer	Stone & Webster
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. Lewis
18.	IE Section Chief	P. J. Kellog
19.	IE Resident Inspector	E. H. Webster (703) 894-5421
20.	Licensing Project Manager	L. Engle
21.	Docket Number	50-338
22.	License Number/Date of Issue	NPF-4, 11/26/77
23.	Public Document Room	Alderman Library/Manuscripts Dept. Univ. of Virginia/Charlottesville VA 22901 & Louisa County Courthouse, Louisa, VA 23093

INSPECTION STATUS

ENFORCEMENT STATUS

NONE.

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

100%.

+ LAST IE SITE INSPECTION DATE: JULY 11, 1980

+ INSPECTION REPORT NO: 50-338/80-29

SUMMARY

INSPECTION SUMMARY

+ INSPECTION JUNE 2 - JULY 11 (80-29): THIS ROUTINE INSPECTION BY THE RESIDENT INSPECTORS INVOLVED 41 HOURS ON SITE IN THE AREAS OF FACILITY OPERATIONS, ADHERENCE TO TECHNICAL SPECIFICATIONS, FIRE EQUIPMENT AND HOUSEKEEPING. OF THE FOUR AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN THE THREE AREAS; ONE APPARENT ITEMS OF NONCOMPLIANCE WAS FOUND IN ONE AREA; (INFRACTION: FAILURE TO COMPLY WITH TECHNICAL SPCIFICATIONS).

REPORTS RECEIVED FROM LICENSEE

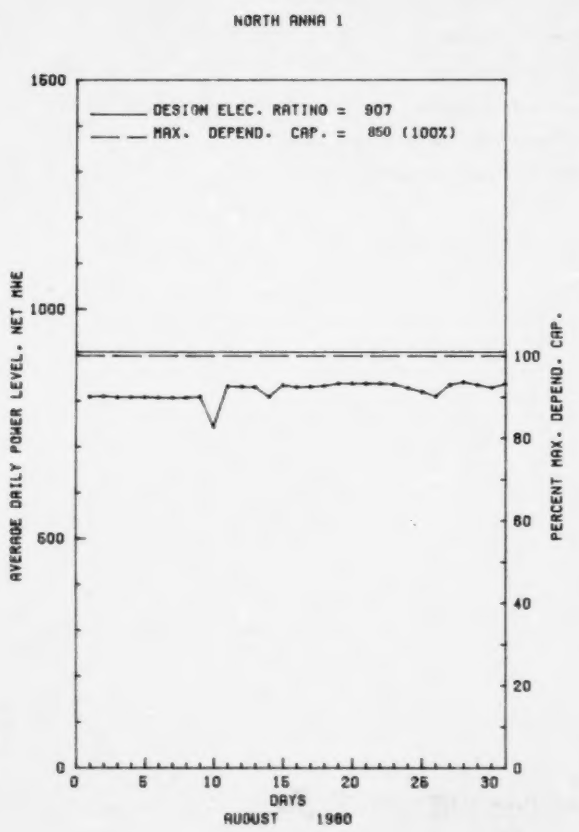
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-055/ 01T-0	06/29/80	07/10/80	NON CONSERVATIVE BORON DILUTION ANALYSIS
80-056/ 03L-0	06/26/80	07/17/80	BAD CARD IN OVERTEMPERATURE DELTA T CIRCUIT CAUSED REACTOR TRIP
80-058/ 03L-0	06/19/80	07/16/80	POST ACCIDENT HYDROGEN MONITOR INOPERATIVE DUE TO LOW PRESSURE IN SAMPLE LINES
80-059/ 03L-0	06/23/80	07/22/80	LHSI RECIRCULATION VALVE FAILED TO OPEN DUE TO FAILED ELECTRICAL SWITCH
80-061/ 01T-0	07/08/80	07/17/80	AFW PUMP NOT CAPABLE OF DELIVERING DESIGN FLOW FOR SOME ACCIDENT CONDITIONS
80-062/ 03L-0	07/08/80	08/04/80	LEAK IN EQUIPMENT HATCH ESCAPE LOCK DUE TO DEFECTIVE BASKET

REVIEWED BY OMPA LOEB: T. C. Cintula **DATE:** 10-7-80

REVIEWED BY NRR DL: **DATE:** 10-7-80

NORTH ANNA 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-338
2. Utility Contact: W. R. Madison (703) 894-5151
3. Licensed Thermal Power (MWt): 2775
4. Nameplate Rating (Gross MWe): 1089 x 0.9 = 980
5. Design Electrical Rating (Net MWe): 907
6. Maximum Dependable Capacity (Gross MWe): 880
7. Maximum Dependable Capacity (Net MWe): - 850
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: MDC LOWERED TO REFLECT A MORE REALISTIC VALUE.
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	5,855.0	19,632.0
12. Hours Reactor Was Critical	744.0	4,975.8	15,145.0
13. Reactor Reserve Shutdown Hours	.0	55.6	203.9
14. Hours Generator On-Line	744.0	4,780.7	14,833.8
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	2,037,456	12,158,081	38,437,994
17. Gross Electrical Energy (MWH)	648,429	3,806,398	12,152,747
18. Net Electrical Energy (MWH)	611,343	3,577,406	11,430,850
19. Unit Service Factor	100.0	81.7	75.6
20. Unit Availability Factor	100.0	81.7	75.6
21. Unit Capacity Factor (MDC Net)	96.7	71.9	68.5
22. Unit Capacity Factor (DER Net)	90.6	67.4	64.7
23. Unit Forced Outage Rate	.0	10.2	10.4

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUELING - 5 WEEKS - DECEMBER 1980 AND JANUARY 1981.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
NONE									

SUMMARY

NORTH ANNA 1 OPERATED AT NEARLY FULL POWER FOR THE ENTIRE MONTH, WITH NO OUTAGES OR REDUCTIONS.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

NORTH ANNA 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1. Location (state/county) (nearest population center): Louisa County, VA; 40 Miles NW of Richmond, VA
2. Electric Reliability Council: Southeastern Electric Reliability Council
3. Type of Reactor: PWR
4. Licensed Power Level (MWT): 2775
5. Design Electrical Rating (MWE Net): 907
6. Date of Initial Criticality: June 12, 1980
7. Date of Commercial Operation: N/A
8. Condenser Cooling Method: Once through
9. Condenser Cooling Water Source: Lake Anna

Utility and Contractor Information

10. Licensee: Virginia Electric & Power Co.
11. Corporate Address: P.O. Box 26666, Richmond, VA 23261
12. Corporate Contact: J. H. Ferguson, Exec. V.P. - Power
13. Architect/Engineer: Stone & Webster
14. Nuclear Steam Supply System: Westinghouse
15. Constructor: Stone & Webster

Regulatory Information

16. IE Region responsible: II
17. IE Branch Chief: R. C. Lewis
18. IE Section Chief: P. J. Kellog
19. IE Resident Inspector: E. H. Webster (703) 894 5421
20. Licensing Project Manager: L. Engle
21. Docket Number: 50-338
22. License Number/Date of Issue: NPF-7, 8/21/80
23. Public Document Room: Alderman Library/Manuscripts Dept. Univ. of Virginia/Charlottesville, VA 22901 & Louisa County Courthouse, Louisa, VA 23093

INSPECTION STATUS

ENFORCEMENT STATUS

NONE.

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

+ CONTINUE POWER ESCALATION TESTING

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

+ POWER ASCENSION TESTING

+ LAST IE SITE INSPECTION DATE: JULY 11, 1980

+ INSPECTION REPORT NO: 50-339/80-28

SUMMARY

INSPECTION SUMMARY

+ INSPECTION JUNE 2 - JULY 11 (80-28): THIS ROUTINE INSPECTION BY THE RESIDENT INSPECTORS INVOLVED 110 HOURS ON SITE IN THE AREAS OF PREVIOUSLY IDENTIFIED OPEN ITEMS, LICENSEE EVENT REPORTS, ITEMS REQUIRING RESOLUTION PRIOR TO ISSUANCE OF A FULL POWER LICENSE AND CONDUCT OF INITIAL CRITICALITY, ZERO POWER AND LOW POWER TESTING. OF THE SEVEN AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED IN ONE AREA (INFRACTION - FAILURE TO ESTABLISH A PROGRAM; INFRACTION - FAILURE TO COMPLY WITH TECHNICAL SPECIFICATIONS).

REPORTS RECEIVED FROM LICENSEE

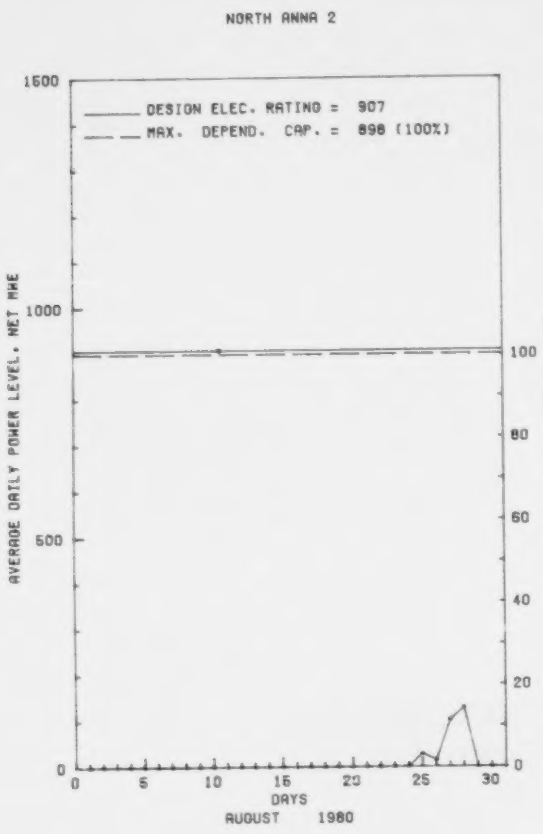
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-032/03L-0	07/06/80	08/04/80	MORE THAN 12 STEP VARIANCE ROD POSITION INDICATOR J13 AND GROUP DEMAND INDICATOR
80-033/03L-0	07/10/80	08/04/80	MORE THAN 12 STEP VARIANCE ROD POSITION INDICATOR N-7 AND GROUP DEMAND INDICATOR
80-034/03L-0	07/11/80	08/07/80	RCS LEAK FROM PACKING FAILURE ON VALVE RHR-MOV-2700
80-035/03L-0	07/17/80	08/08/80	RCS LEAK OF 1.48 GPM ON "A" RCP SEAL INJECTION LINE
80-039/03L-0	07/13/80	08/07/80	PRESSURIZER PROTECTION CHANNEL III OUT OF CALIBRATION
80-041/03L-0	07/24/80	08/07/80	MORE THAN 12 STEP VARIANCE ROD POSITION INDICATOR G-13 AND GROUP DEMAND INDICATOR
80-042/03L-0	07/26/80	08/07/80	MORE THAN 12 STEP VARIANCE ROD POSITION INDICATOR E5 AND GROUP DEMAND INDICATOR

REVIEWED BY OMPA LOEB: T. C. Cintula DATE: 10.7.80

REVIEWED BY NRR DL: BJ Youngblood DATE: [illegible]

NORTH ANNA 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-339
2. Utility Contact: W.A. WOODSMALL, III (7030 894-5151
3. Licensed Thermal Power (MWt): 2775
4. Nameplate Rating (Gross MWe): 947
5. Design Electrical Rating (Net MWe): 907
6. Maximum Dependable Capacity (Gross MWe): 928
7. Maximum Dependable Capacity (Net MWe): 898
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: ITEM 3 CHANGED TO 2775 BY FULL-POWER LICENSE 8/21/80
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	2,208.0	2,208.0
12. Hours Reactor Was Critical	225.8	736.4	736.4
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	55.0	55.0	55.0
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	39,455	43,817	43,817
17. Gross Electrical Energy (MWH)	8,174	8,174	8,174
18. Net Electrical Energy (MWH)	6,562	6,562	6,562

19. Unit Service Factor
20. Unit Availability Factor
21. Unit Capacity Factor (MDC Net)
22. Unit Capacity Factor (DER Net)
23. Unit Forced Outage Rate

NOT IN COMMERCIAL OPERATION

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): TMI MODS - 11/01/80; MAINTENANCE - 04/01/81.
25. If Shut Down At End of Report Period, Estimated Date of Startup: 09/04/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause and Corrective Action to Prevent Recurrence
80-01	08/26/80	F	18.1	H	3				TURBINE TRIP/REACTOR TRIP WHEN GENERATOR BREAKER WAS OPENED. PLACED RECORDERS ON REACTOR PROTECTION CIRCUITS TO TRACE CAUSE.
80-02	08/27/80	F	2.4	H	3				TURBINE TRIP/REACTOR TRIP WHEN OVERSPEED PROTECTION CONTROLLER WAS ACTIVATED DURING TEST.
80-03	08/28/80	F	2.0	A	3				STEAM GENERATOR LO LEVEL REACTOR TRIP CAUSED BY LOSS OF FEEDWATER FLOW WHEN CONDENSATE PUMP SUCTION STRAINERS WERE CLOGGED UP. CLEANED STRAINERS.
80-04	08/28/80	F	79.5	D	1	80-51	HH	VALVEX	UNIT SHUTDOWN TO COMPLY WITH TECHNICAL SPECIFICATIONS 3.6.3.1 FEEDWATER/CONTAINMENT PENETRATION ISOLATION.

SUMMARY

NORTH ANNA 2 WAS GRANTED A FULL POWER LICENSE ON AUGUST 21, 1980 AND ARE IN POWER ASCENSION TESTING.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & Licensee Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source

OCONEE 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1. Location (state/countv) (nearest population center)	South Carolina/Oconee 30 miles W of Greenville
2. Electric Reliability Council	Southeastern Electric Reliability Council
3. Type of Reactor	PWR
4. Power Level (MWT)	2568
5. Design Electrical Rating (MWE Net)	887
6. Date of Initial Criticality	April 19, 1973
7. Date of Commercial Operation	July 15, 1973
8. Condenser Cooling Method	Once Through
9. Condenser Cooling Water Source	Keowee Lake

Utility & Contractor Information

10. Licensee	Duke Power Company
11. Corporate Address	422 S. Church Street Charlotte, North Carolina 28242
12. Corporate Contact	W. O. Parker, Jr.
13. Architect/Engineer	DPC and Bechtel (Containment Bldg.)
14. Nuclear Steam Supply System	B&W
15. Constructor	DPC

Regulatory Information

16. IE Region responsible	II
17. IE Branch Chief	R.C. Lewis
18. IE Section Chief	R. MARTIN
19. IE Resident Inspector	F. Jape (803) 882 6927
20. Licensing Project Manager	M. B. Fairtile
21. Docket Number	50-269
22. License Number/Date of Issue	DPR-38, 2/6/73
23. Public Document Room	Oconee County Library 201 S. Spring Street Walhalla, South Carolina 29691

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

UNIT SHUTDOWN AT END OF MONTH.

LAST IE SITE INSPECTION DATE: AUGUST 1-31, 1980

INSPECTION REPORT NO: 50-269/80-30

SUMMARY

INSPECTION SUMMARY

NO REPORTS RELEASED THIS MONTH.

REPORTS RECEIVED FROM LICENSEE

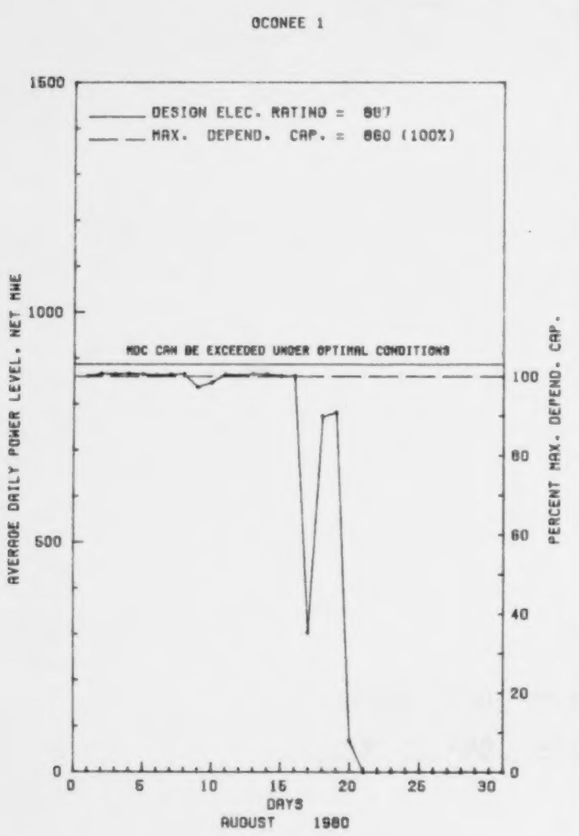
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
ETS 80-1	07/10/80	07/16/80	HIGH PH EXCURSION INTO OIL COLLECTION BASIN
80-019/ 03L	06/18/80	07/17/80	INOP CONCENTRATED BORIC ACID STORAGE TANK PUMP
80-020/ 01T	06/19/80	07/03/80	BACK-UP 125VDC POWER WAS REMOVED FROM SERVICE FOR LONGER THAN TS 24-HOURS
80-021/ 03L	06/12/80	07/11/80	OVERHEATING "B" LOOP HPI LINES DUE TO LEAKAGE PAST CHECK VALVE
80-022/ 03L	07/07/80	08/06/80	"B" HPSW PUMP REMOVED FROM SERVICE DUE TO LEAKAGE FROM PUMPS MOTOR COOLER
80-023/ 01T-0	07/09/80	08/05/80	DURING STARTUP AFTER MAINTENANCE THE ACTUAL CRITICAL CONFIGURATION WAS DIFFERENT FROM THE PREDICTED

REVIEWED BY OMPA LOEB: John L Crooks **DATE:** 10/6/80

REVIEWED BY NRR DL: M.B. Fairtile **DATE:** 10/7/80

OCONEE 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-269
2. Utility Contact: J. A. Reavis (704) 373-8552
3. Licensed Thermal Power (MWt): 2568
4. Nameplate Rating (Gross MWe): 1038 x 0.9 = 934
5. Design Electrical Rating (Net MWe): 887
6. Maximum Dependable Capacity (Gross MWe): 899
7. Maximum Dependable Capacity (Net MWe): 860-
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

CAPACITY FACTORS CALCULATED WITH A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	5,855.0	62,496.0
12. Hours Reactor Was Critical	464.0	3,941.9	44,[illegible]56.6
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	450.7	3,836.9	41,784.0
15. Unit Reserve Shutdown Hours	.0	.0	.3
16. Gross Thermal Energy (MWH)	1,127,186	8,398,654	97,616,773
17. Gross Electrical Energy (MWH)	396,190	2,974,070	33,888,370
18. Net Electrical Energy (MWH)	374,441	2,818,378	32,049,877
19. Unit Service Factor	60.6	65.5	66.9
20. Unit Availability Factor	60.6	65.5	66.9
*21. Unit Capacity Factor (MDC Net)	58.5	56.0	59.4
22. Unit Capacity Factor (DER Net)	56.7	54.3	57.8
23. Unit Forced Outage Rate	39.4	14.1	17.7

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
TMI RELATED MODIFICATION - NOV. 17 - 5 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: 09/02/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
6	08/17/80	F	11.2	A	3		HA	TURBIN	A BAD CARD IN THE EHC CONTROL SYSTEM CAUSED A TURBINE/ REACTOR TRIP DUE TO LOW EHC OIL PRESSURE.
7	08/20/80	F	282.1	A	1		CH	HTEXCH	TUBE LEAKS IN THE 1B1 FEEDWATER HEATER CAUSED A SHUTDOWN WHEN THE HEATER COULD NOT BE ISOLATED SUFFICIENTLY.

SUMMARY

OCONEE 1 OPERATED AT FULL POWER UNTIL AUGUST 17TH WHEN THE PLANT ENCOUNTERED TWO EQUIPMENT FAILURE OUTAGES.

LEGEND

[1] Type:
F-Forced
S-Scheduled

[2] Reason:
A-Equipment Failure
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error
H-Other

[3] Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Continuations
5-Load Reduction
9-Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

OCONEE 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	South Carolina/Oconee 30 miles W of Greenville
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2568
5.	Design Electrical Rating (MWE Net)	887
6.	Date of Initial Criticality	November 11, 1973
7.	Date of Commercial Operation	September 9, 1974
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Keowee Lake

Utility & Contractor Information

10.	Licensee	Duke Power Company
11.	Corporate Address	422 S. Church Street Charlotte, North Carolina 28242
12.	Corporate Contact	W. O. Parker, Jr.
13.	Architect/Engineer	DPC/Bechtel
14.	Nuclear Steam Supply System	B&W
15.	Constructor	DPC

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R.C. Lewis
18.	IE Section Chief	R. MARTIN
19.	IE Resident Inspector	F. Jape (803) 882-6927
20.	Licensing Project Manager	M. B. Fairtile
21.	Docket Number	50-270
22.	License Number/Date of Issue	DPR-47, 10/6/73
23.	Public Document Room	Oconee County Library 201 S. Spring Street Walhalla, South Carolina 29691

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO TS 6.4.1.J, ON JUNE 11, 1980 THE OCONEE UNIT 2, TURBINE DRIVEN EMERGENCY FEEDWATER PUMP AND ASSOCIATED EQUIPMENT WAS TEST OPERATED USING AN OCONEE UNIT 1 OPERATING PROCEDURE. CONTRARY TO TS 6.4.1.J ON JUNE 12, 1980, THE OCONEE UNIT 2 TURBINE DRIVEN EMERGENCY FEEDWATER PUMP AND ASSOCIATED EQUIPMENT WAS TEST OPERATED EMPLOYING VALVE LINE-UP CONFIGURATION CONTRARY TO THE REQUIREMENTS OF THE WRITTEN APPROVED PROCEDURE.
(8020 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE

FACILITY ITEMS (PLANS AND PROCEDURES):

FIVE WEEK MAINTENANCE OUTAGE SCHEDULED TO BEGIN IN MID OCTOBER, 1980.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

OPERATION AT REDUCED POWER

LAST IE SITE INSPECTION DATE: AUGUST 1-31, 1980

INSPECTION REPORT NO: 50-270/80-26

SUMMARY

INSPECTION SUMMARY

NO REPORTS RELEASED THIS MONTH.

REPORTS RECEIVED FROM LICENSEE

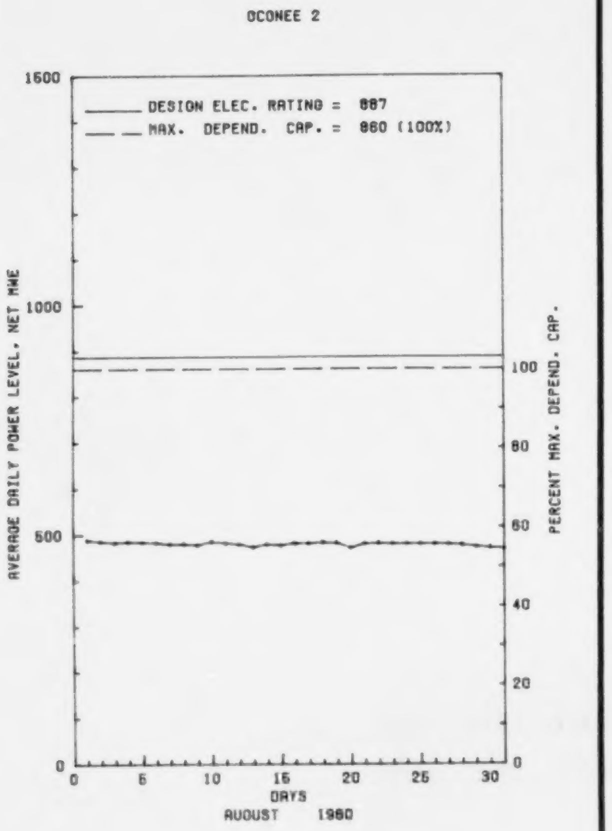
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
ETS 80-1	07/10/80	07/16/80	HIGH PH EXCURSION INTO OIL COLLECTION BASIN
80-006/01T	07/25/80	07/25/80	LOW PRESSURE INJECTION SYSTEM VALVE (MOTOR OPERATED) FAILED TO OPERATE DURING SURVEILLANCE
80-008/01L-0	07/13/80	08/12/80	THE NUMBER 2 HIGH PRESSURE INJECTION WAS REMOVED FROM SERVICE DUE TO A BAD BEARING

REVIEWED BY OMPA LOEB: John R Crooks **DATE:** 10/6/80

REVIEWED BY NRR DL: M. B. Fairtile **DATE:** 10/7/80

OCONEE 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-270
2. Utility Contact: J. A. Reavis (704) 373-8552
3. Licensed Thermal Power (MWt): 2568
4. Nameplate Rating (Gross MWe): 1038 x 3.9 = 934
5. Design Electrical Rating (Net MWe): 887
6. Maximum Dependable Capacity (Gross MWe): 899
7. Maximum Dependable Capacity (Net MWe): 860

CAPACITY FACTORS CALCULATED WITH A WEIGHTED AVERAGE

8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	5,855.0	52,416.0
12. Hours Reactor Was Critical	744.0	3,244.0	36,840.0
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	744.0	3,144.2	35,920.3
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,159,057	6,511,637	86,559,041
17. Gross Electrical Energy (MWH)	372,210	2,209,280	28,723,636
18. Net Electrical Energy (MWH)	356,095	2,081,234	27,244,992
19. Unit Service Factor	100.0	53.7	68.5
20. Unit Availability Factor	100.0	53.7	68.5
*21. Unit Capacity Factor (MDC Net)	55.7	41.3	60.1
22. Unit Capacity Factor (DER Net)	54.0	40.1	58.6
23. Unit Forced Outage Rate	.0	3.1	18.8

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
TMI RELATED MODIFICATION - OCTOBER 12 - 5 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
12-P	08/01/80	F	.0	B	5		SF	PUMPXX	MAINTENANCE PROBLEMS ON THE 2 "B" HIGH PRESSURE INJECTION PUMP REQUIRED THE UNIT TO STAY AT REDUCED POWER PER TECH. SPEC. FOR OPERATION WITH TWO (2) PUMPS.

SUMMARY

OCONEE 2 OPERATED AT REDUCED POWER FOR THE MONTH DUE TO HIGH PRESSURE INJECTION PUMP PROBLEMS.

LEGEND

[1] Type:
F-Forced
S-Scheduled

[2] Reason:
A-Equipment Failure
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error
H-Other

[3] Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Continuations
5-Load Reduction
9-Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

OCONEE 3

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	South Carolina/Oconee 30 miles W of Greenville
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2568
5.	Design Electrical Rating (MWE Net)	887
6.	Date of Initial Criticality	September 5, 1974
7.	Date of Commercial Operation	December 16, 1974
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Keowee Lake

Utility & Contractor Information

10.	Licensee	Duke Power Company
11.	Corporate Address	422 S. Church Street Charlotte, North Carolina 28242
12.	Corporate Contact	W. O. Parker, Jr.
13.	Architect/Engineer	DPC/Bechtel
14.	Nuclear Steam Supply System	B&W
15.	Constructor	DPC

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R.C. Lewis
18.	IE Section Chief	R. MARTIN
19.	IE Resident Inspector	F. Jape (803) 882-6927
20.	Licensing Project Manager	M. B. Fairtile
21.	Docket Number	50-287
22.	License Number/Date of Issue	DPR-55, 7/19/74
23.	Public Document Room	Oconee County Library 201 S. Spring Street Walhalla, South Carolina 29691

INSPECTION STATUS

ENFORCEMENT STATUS

AS REQUIRED BY TECHNICAL SPECIFICATION 6.4.1.C, THE STATION SHALL BE OPERATED AND MAINTAINED IN ACCORDANCE WITH APPROVED PROCEDURES. WRITTEN PROCEDURES WITH APPROPRIATE CHECK-OFF LISTS AND INSTRUCTIONS SHALL BE PROVIDED FOR ACTIONS TAKEN TO CORRECT SPECIFIC AND FORESEEN POTENTIAL MALFUNCTIONS OF SYSTEMS OR COMPONENTS INVOLVING NUCLEAR SAFETY AND RADIATION LEVELS. INCLUDING RESPONSES TO ALARMS. SUSPECTED PRIMARY SYSTEM LEAKS AND ABNORMAL REACTIVITY CHANGES. CONTRARY TO THE ABOVE. ADEQUATE PROCEDURES DID NOT EXIST TO GUIDE THE OPERATOR PROPERLY DURING THE LOSS OF MOST CONTROL BOARD INDICATIONS, WHICH OCCURRED ON NOVEMBER 10, 1979, AS A RESULT OF LOSS OF THE KI INVERTER POWER SUPPLY FOR THE INTEGRATED CONTROL SYSTEM AND MOST CONTROL BOARD INDICATIONS. PROCEDURES WERE ALSO INADEQUATE TO GUIDE THE OPERATOR DURING RECOVERY FROM THIS EVENT IN THAT MAXIMUM ALLOWABLE REACTOR COOLANT SYSTEM COOLDOWN RATES WERE EXCEEDED.
(7937 INFRACTION)

AS REQUIRED BY TECHNICAL SPECIFICATION 3.1.2.1. OCONEE UNIT 3 REACTOR COOLANT SYSTEM COOLDOWN RATES SHALL BE LIMITED IN ACCORDANCE WITH FIGURE 3.1.2-2.C. THIS FIGURE SPECIFIES THAT. THE MAXIMUM COOLDOWN RATE BETWEEN 532 DEGREES F AND 432 DEGREES F WILL BE LESS THAN OR EQUAL TO 100 DEGREES F PER HOUR. CONTRARY TO THE ABOVE, DURING RECOVERY FROM THE TRANSIENT EVENT ON NOVEMBER 10, 1979, THE UNIT 3 REACTOR COOLANT SYSTEM WAS COOLED DOWN APPROXIMATELY 140 DEGREES F. WITHIN THIRTY MINUTES. FROM AN INDICATED 560 DEGREES F TO AN INDICATED 420 DEGREES F.
(7937 INFRACTION)

ITEM 6 OF IE BULLETIN 79-05B. DATED APRIL 21, 1979, REQUIRED THAT YOU REVIEW YOUR PROMPT REPORTING PROCEDURES FOR NRC NOTIFICATION TO ASSURE THAT NRC IS NOTIFIED WITHIN ONE HOUR OF THE TIME THE REACTOR IS NOT IN A CONTROLLED OR EXPECTED CONDITION OF OPERATION. FURTHER, AT THAT TIME AN OPEN CONTINUOUS COMMUNICATION CHANNEL SHALL BE ESTABLISHED AND MAINTAINED WITH NRC. CONTRARY TO THE ABOVE, AT 3:15 P.M., ON NOVEMBER 10, 1979, OCONEE UNIT 3 EXPERIENCED A TRANSIENT DURING WHICH THE LOSS OF THE KI INVERTER RESULTED IN THE UNIT BEING IN AN UNEXPECTED CONDITION OF OPERATION, AND THE NRC WAS NOT NOTIFIED UNTIL APPROXIMATELY 8:30 P.M. ON THE SAME DATE.
(7937 DEVIATION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

REFUELING OUTAGE SCHEDULED TO BEGIN 11/80.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

OPERATING AT FULL POWER.

LAST IE SITE INSPECTION DATE: AUGUST 1-31, 1980

INSPECTION REPORT NO: 50-287/80-23

SUMMARY

INSPECTION SUMMARY

NO REPORTS RELEASED THIS MONTH.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

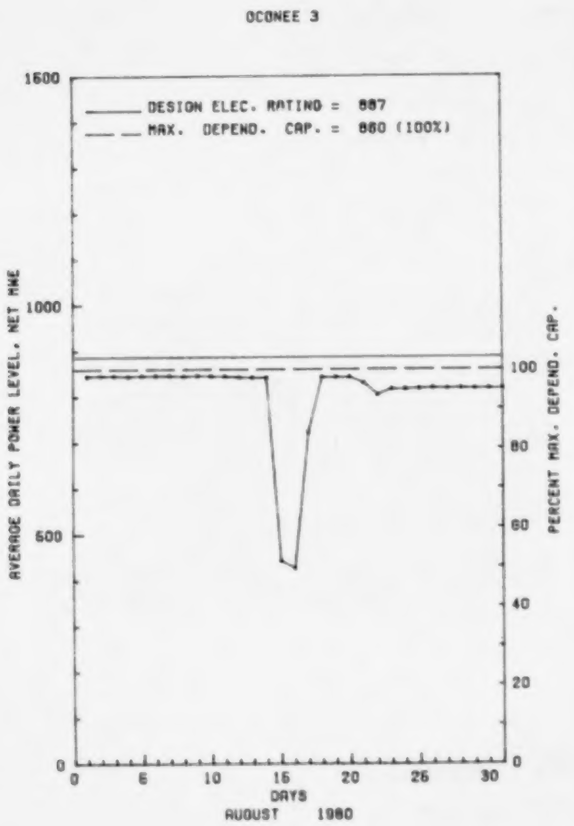
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
ETS 80-1	07/10/80	07/16/80	HIGH PH EXCURSION INTO OIL COLLECTION BASIN
80-009/01T	07/11/80	07/24/80	2 CHANNELS OF EM POWER SWITCHING LOGIC OUT OF SERVICE DEFEATING LOAD SHED AND TRANSFER TO STANDBY
80-010/01T	06/14/80	07/11/80	OTSG TUBE FAILURE
80-011/01T	06/25/80	07/16/80	CRACKED MANWAY STUDS - 3A OTSG

REVIEWED BY OMPA LOEB: John R. Crooks DATE: 10/6/80

REVIEWED BY NRR DL: M. B. Fairtile DATE: 10/7/80

OCONEE 3

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-287
2. Utility Contact: J. A. Reavis (704) 373-8552
3. Licensed Thermal Power (MWt): 2568
4. Nameplate Rating (Gross MWe): 1038 x 0.9 = 934
5. Design Electrical Rating (Net MWe): 887
6. Maximum Dependable Capacity (Gross MWe): 899
7. Maximum Dependable Capacity (Net MWe): 860
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

CAPACITY FACTORS CALCULATED WITH A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	5,855.0	50,063.0
12. Hours Reactor Was Critical	742.4	4,306.6	36,200.5
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	731.9	4,224.1	35,286.0
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,825,755	10,627,862	85,509,467
17. Gross Electrical Energy (MWH)	627,380	3,669,620	29,620,884
18. Net Electrical Energy (MWH)	598,692	3,496,626	27,943,184
19. Unit Service Factor	98.4	72.1	70.5
20. Unit Availability Factor	98.4	72.1	70.5
*21. Unit Capacity Factor (MDC Net)	93.6	69.4	64.7
22. Unit Capacity Factor (DER Net)	90.7	67.3	63.0
23. Unit Forced Outage Rate	1.6	13.4	17.6

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
*: REFUELING & SSF TIE-IN - DECEMBER 21 - 15 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
5	08/15/80	F	6.5	A	3		EB	CKTBRK	UNIT TRIPPED DUE TO POWER LOSS TO TURBINE EHC PUMPS WHEN MCC-3XA TRIPPED.
5A	08/15/80	F	5.6	H	9		RC	FUELXX	REACTOR QUADRANT POWER TILT CAUSED A HOLD IN STARTUP.
9-P	08/16/80	F	.0	A	5		CH	VALVEX	HOLDING AT 60% POWER FOR REPAIRS TO "B" FEEDWATER SUCTION RELIEF VALVE.
10-P	08/21/80	F	.0	B	5		CH	TURBIN	3A FEEDWATER PUMP TURBINE TRIPPED WHILE PERFORMING FDWPT OIL TRIP TEST.
11-P	08/22/80	F	.0	A	5		ZZ	ZZZZZZ	HOLDING AT 96% POWER FOR BETTER UNIT CONT WITH RPS CHANNELS A AND D NOT SERVICEABLE.

SUMMARY

OCONEE 3 OPERATED AT NEAR FULL POWER FOR THE BULK OF THE MONTH, WITH THE EXCEPTION OF TWO OUTAGES AND THREE REDUCTIONS LISTED IN DETAIL ABOVE.

LEGEND

[1] Type:
- F-Forced
- S-Scheduled

[2] Reason:
- A-Equipment Failure
- B-Maintenance or Test
- C-Refueling
- D-Regulatory Restriction
- E-Operator Training & License Examination
- F-Administrative
- G-Operational Error
- H-Other

[3] Method:
- 1-Manual
- 2-Manual Scram
- 3-Automatic Scram
- 4-Continuations
- 5-Load Reduction
- 9-Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

OYSTER CREEK

FACILITY DATA

Facility Description		
1.	Location (state/county) (nearest population center)	New Jersey/Ocean 9 miles S of Toms River
2.	Electric Reliability Council	Mid-Atlantic Area Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	1930
5.	Design Electrical Rating (MWE Net)	650
6.	Date of Initial Criticality	May 3, 1969
7.	Date of Commercial Operation	December 1969
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Barnegat Bay
Utility & Contractor Information		
10.	Licensee	Jersey Central Power & Light Co.
11.	Corporate Address	Madison Avenue At Punch Bowl Road Morristown, New Jersey 07960
12.	Corporate Contact	I. Finfrock, Vice President Power Generation
13.	Architect/Engineer	Burns & Roe
14.	Nuclear Steam Supply System	GE
15.	Constructor	J. A. Jones
Regulatory Information		
16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	IE Section Chief	R. R. KEIMIG
19.	IE Resident Inspector	L. BRIGGS (X5306) (609) 693-0702 NRC RESIDENT
20.	Licensing Project Manager	W. A. Paulson
21.	Docket Number	50-219
22.	License Number/Date of Issue	DPR-16, 4/9/69
23.	Public Document Room	Ocean County Library 15 Hooper Avenue Toms River, New Jersey 08753

INSPECTION STATUS

ENFORCEMENT STATUS

10 CFR 20.103, "EXPOSURE OF INDIVIDUALS TO CONCENTRATIONS OF RADIOACTIVE MATERIALS IN AIR IN RESTRICTED AREAS" STATES IN PARAGRAPH (B) (IN PART), "THE LICENSEE SHALL, AS A PRECAUTIONARY PROCEDURE, USE PROCESS OR OTHER ENGINEERING CONTROLS, TO THE EXTENT PRACTICABLE, TO LIMIT CONCENTRATIONS OF RADIOACTIVE MATERIALS IN AIR TO LEVELS BELOW THOSE WHICH DELIMIT AN AIRBORNE RADIOACTIVITY AREA AS DEFINED IN 20.203(D)(1)(II)... WHEN IT IS IMPRACTICABLE TO APPLY PROCESS OR OTHER ENGINEERING CONTROLS...OTHER PRECAUTIONARY PROCEDURES, SUCH AS INCREASED SURVEILLANCE, LIMITATION OF WORKING TIMES OR THE PROVISION OF RESPIRATORY PROTECTIVE EQUIPMENT, SHALL BE USED TO MAINTAIN INTAKE OF RADIOACTIVE MATERIALBY ANY INDIVIDUAL WITHIN ANY PERIOD OF SEVEN CONSECUTIVE DAYS AS FAR BELOW THAT INTAKE WHICH WOULD RESULT FROM INHALATION OF SUCH MATERIAL FOR 40 HOURS AT THE UNIFORM CONCENTRATIONS SPECIFIED IN APPENDIX B, TABLE 1, COLUMN 1." CONTRARY TO THE ABOVE, NO PROCESS OR ENGINEERING CONTROLS, TO THE EXTENT PRACTICABLE OR OTHER PRECAUTIONARY PROCEDURES WERE USED BY THE LICENSEE DURING CLEANING AND EXAMINATION OF CONTROL ROD BLADE HANDLING TOOLS ON MARCH 2, 1980 TO LIMIT CONCENTRATIONS OF RADIOACTIVE MATERIALS IN AIR AND INTAKE OF RADIOACTIVE MATERIAL BY TWO INDIVIDUALS PERFORMING THE ABOVE EVALUATIONS. THE INDIVIDUALS INVOLVED IN THE ABOVE SUSTAINED EXTENSIVE RADIOACTIVE SKIN CONTAMINATION AND INTAKES OF RADIOACTIVE MATERIAL IN EXCESS OF THE 7 DAY INTAKE LIMIT.
(8511 INFRACTION)

10 CFR 20.103(A)(3) STATES IN PART, "FOR PURPOSES OF DETERMINING COMPLIANCE WITH THE REQUIREMENTS OF THIS SECTION, THE LICENSEE SHALL USE SUITABLE MEASUREMENTS OF CONCENTRATIONS OF RADIOACTIVE MATERIALS IN AIR, FOR DETECTING AND EVALUATING AIRBORNE RADIOACTIVITY IN RESTRICTED AREAS...." CONTRARY TO THE ABOVE, ON MARCH 2, 1980, AT 10:00 AM AND 8:20 PM RESPECTIVELY, TWO CONTROL ROD BLADE HANDLING TOOLS WERE REMOVED FROM THE FACILITIES SPENT FUEL STORAGE POOL, HANDLED, EXAMINED, AND FOR AT LEAST ONE TOOL, REPAIRED BY PERSONNEL WITHOUT SUITABLE MEASUREMENTS OF AIRBORNE RADIOACTIVITY BEING MADE. THE TOOLS EXHIBITED RADIOACTIVE CONTAMINATION OF 345,300 DPM BETA-GAMMA AND 47 DPM ALPHA PER 100 CENTIMETERS SQUARED AND 850 MILLIRAD PER HOUR BETA AND 190 DPM ALPHA PER 100 CENTIMETERS SQUARED RESPECTIVELY.
(8511 INFRACTION)

TECHNICAL SPECIFICATION 6.8, PROCEDURES REQUIRES IN SECTION 6.8.1, THAT WRITTEN PROCEDURES BE ESTABLISHED, IMPLEMENTED, AND MAINTAINED THAT MEET OR EXCEED THE REQUIREMENTS OF SECTION 5.1 AND 5.3 OF AMERICAN NATIONAL STANDARD N18.7-1972 AND APPENDIX "A" OF REGULATORY GUIDE 1.33-1972. APPENDIX "A" OF GUIDE 1.33 LISTS, IN SECTION G.5.E, RADIOACTIVE WORK PERMIT PROCEDURE AS A PROCEDURE TO BE PREPARED.(ANSI) N18.7 PRESENTS IN SECTION 5.3.2 THE CONTENT TO BE INCLUDED IN A PROCEDURE. INCLUDED IN 5.3.2 ARE PRECAUTIONS, LIMITATIONS AND ACTIONS. CONTRARY TO THE ABOVE, FROM JULY 2, 1979 TO MARCH 18, 1980, NO RWP PROCEDURE WAS ESTABLISHED, IMPLEMENTED OR MAINTAINED MEETING THE ABOVE REQUIREMENTS IN THAT THE PROCEDURE ESTABLISHED BY THE LICENSEE AS A PROCEDURE DID NOT CONTAIN PRECAUTIONS OR LIMITATIONS AND ACTIONS. THE RWP PROCEDURE DID NOT CONTAIN: (1) PRECAUTIONS, LIMITATIONS OR ACTIONS TO BE TAKEN BY PERSONNEL, INCLUDING RADIATION PROTECTION TECHNICIAN ACTIONS, SUCH AS TERMINATION DUE TO CHANGING RADIATION CONDITIONS, ACTIONS TO BE TAKEN AS A RESULT OF A WORKER'S FAILURE TO ADHERE TO THE REQUIREMENTS; (2) CRITERIA TO DEFINE THE CONDITIONS FOR USE OF EXTENDED VERSUS ROUTINE RWP; (3) LIMITATIONS REGARDING THE USE OF ONE RWP TO COVER MULTIPLE WORK ACTIVITIES; (4) REQUIRED ACTION (BASED ON RADIOLOGICAL HAZARDS PRESENT) SUCH AS HOLDING A PREPLANNING MEETING PRIOR TO COMMENCEMENT OF THE WORK. THE PREPLANNING MEETINGS WOULD SERVE TO INSTRUCT WORKERS IN THE PRECAUTIONS AND PROCEDURES TO MINIMIZE EXPOSURE; AND, (5) CONDUCT OF AN AS LOW AS REASONABLY ACHIEVABLE (ALARA) REVIEW PRIOR TO COMMENCEMENT OF WORK.
(8511 VIOLATION)

TECHNICAL SPECIFICATION 6.11, "RADIATION PROTECTION PROGRAM," STATES, "PROCEDURES FOR PERSONNEL RADIATION PROTECTION SHALL BE PREPARED CONSISTENT WITH THE REQUIREMENTS OF 10 CFR 20 AND SHALL BE APPROVED, MAINTAINED AND ADHERED TO FOR ALL OPERATIONS INVOLVING PERSONNEL RADIATION EXPOSURE." RADIATION PROTECTION PROCEDURE NO. 915.7, REVISION 5, "PERSONNEL MONITORING," DEVELOPED PURSUANT TO THE ABOVE, REQUIRES IN PARAGRAPH 5.6 THAT EXTREMITY DOSIMETRY SHALL BE ISSUED WHEN THE ESTIMATED EXTREMITY DOSE IS EXPECTED TO EXCEED FOUR (4) TIMES THE ESTIMATED WHOLE BODY DOSE. ADDITIONALLY, 10 CFR 20.202(A) REQUIRES PERSONNEL MONITORING EQUIPMENT BE SUPPLIED BY THE LICENSEE AND USED BY EACH INDIVIDUAL WHO IS LIKELY TO RECEIVE 25 PERCENT OF THE APPLICABLE VALUE SPECIFIED IN 10 CFR 20.101(A). CONTRARY TO THE ABOVE, ON MARCH 2, 1980, PERSONNEL PERFORMED WORK ON A CONTROL ROD BLADE HANDLING TOOL, WHICH EXHIBITED EXTREMITY DOSES APPROXIMATELY 40 TIMES THE WHOLE BODY DOSE AND PERSONNEL WERE LIKELY TO RECEIVE 25 PERCENT OF THE APPROPRIATE LIMIT SPECIFIED IN 20.101(A) AND EXTREMITY DOSIMETRY WAS NOT ISSUED TO PERSONNEL PERFORMING WORK ON THE TOOL.
(8511 INFRACTION)

CONTINUED ON PAGE O9-A

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-002E	11/25/79	07/29/80	LESS THAN TWO DILUTION PUMPS IN OPERATION WHEN WATER TEMP. WAS LESS THAN 60 DEGREES F. FAILURE DISCOVERED DURING QA AUDIT ON JUNE 17, 1980.
80-026/ 03L	07/09/80	08/08/80	FAILURE OF ONE HYDRAULIC SNUBBER TO LOCK UP DURING TESTING. PLANT WAS IN COLD SHUTDOWN.
80-027/ 01P	07/16/80	07/17/80	REACTOR BLDG TO SUPPRESSION CHAMBER VACUUM BREAKER SYSTEM INLET PIPE FOUND TO BE BLOCKED BY A PLASTIC BAG. VIOLATION OF T.S. 3.5.A.4.A.
80-029/ 03L	07/11/80	08/11/80	FAILURE OF HIGH DRYWELL PRESSURE SWITCH AND SUBSEQUENT INITIATION OF CORE SPRAY SYSTEM - NO INJECTION. RESULTED IN MANUAL DEFEAT OF BOTH CORE SPRAY SYSTEMS AND PLANT SHUTDOWN.
80-03E	07/20/80	07/30/80	FAILURE TO RUN SECOND DILUTION PUMP WHEN BRIDGE TEMPERATURE WAS ABOVE 87 DEGREES F.
80-030/ 03L	07/16/80	08/11/80	FAILURE OF ONE ELECTROMATIC RELIEF VALVE TO OPERATE DURING OPERABILITY TESTING.
80-031/ 03L	07/16/80	08/11/80	FAILURE OF ONE HYDRAULIC SNUBBER TO LOCK UP DURING TESTING REPLACED WITH OPERABLE SPARE.
80-032/ 01T	07/16/80	08/14/80	BOTH WATERTIGHT DOORS TO CONTAINMENT SPRAY PUMP ROOMS FOUND OPEN.
80-04E	07/27/80	08/07/80	LOSS OF DILUTION PUMPS, SEVEN TIMES, OVER A THREE DAY PERIOD DUE TO HIGH LUBE OIL TEMPERATURE TRIPS

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

THE REACTOR WAS SHUTDOWN ON JULY 31 TO REPAIR LEAKS FROM THE CONTROL NITROGEN SYSTEM INSIDE THE DRYWELL. PLANT OPERATIONS WERE RESUMED ON AUGUST 4, 1980 AFTER LOCATING AND REPAIRING A LEAK ON THE NITROGEN SUPPLY TO AN MSIV ACCUMULATOR.

PLANT STATUS:

PLANT IS OPERATING AT 88 % POWER WITH AN OFFGAS RATE OF 71,000 MICROCURIES PER SECOND AND A STACK RELEASE RATE OF 1160 MICROCURIES PER SECOND.

LAST IE SITE INSPECTION DATE: 8/4-29/80

INSPECTION REPORT NO: 50-219/80-28

SUMMARY

INSPECTION SUMMARY

50-219/80-19 & 80-21 - MAY 1-30: ROUTINE INSPECTION BY TWO RESIDENT INSPECTORS AND ONE REGION-BASED CONSTRUCTION BRANCH INSPECTOR (132 HRS) OF: LICENSEE ACTION ON PREVIOUS INSPECTION FINDINGS; FOLLOWUP ON COMMITMENTS MADE IN RESPONSE TO PERFORMANCE APPRAISAL BRANCH INSPECTION; REVIEW OF ONSITE EVENTS THAT OCCURRED DURING THE INSPECTION; REFUELING ACTIVITIES; REVIEW OF LICENSEE ACTION TO PLACE ADMINISTRATIVE CONTROLS ON OVERRIDE OF SAFETY ACTUATION SIGNALS DURING CONTAINMENT PURGING; FOLLOWUP ON REGIONAL REQUESTS; REVIEW OF WOODWARD GOVERNOR OPERATING PROCEDURES; FIRE PROTECTION SYSTEM INSTALLATION REVIEW; AND, INVESTIGATION BY TWO INVESTIGATORS ON MAY 22-23, 1980 (32 HRS) INTO THE CIRCUMSTANCES SURROUNDING A LICENSEE REPORTED EVENT CONCERNING BYPASSED INTERLOCKS IN THE CONTROL ROD DRIVE SYSTEM. TWO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED: FAILURE TO FOLLOW PLANT PROCEDURES; INADEQUATE PROCEDURE CONCERNING BYPASSED REFUELING INTERLOCKS.

50-219/80-23 - MAY 28 - JUL 11: SPECIAL, ANNOUNCED INSPECTION BY ONE REGION-BASED INSPECTOR (230 HRS) OF THE RADIATION PROTECTION PROGRAM, AND ROUTINE UNANNOUNCED INSPECTION BY A RESIDENT INSPECTOR (60 HRS) OF THE EXTENDED PLANT OUTAGE. AREAS INSPECTED WERE: ACTIONS ON PREVIOUS INSPECTION FINDINGS AND COMMITMENTS TO THE DIRECTOR, IE, AND TO THE DIRECTOR, REGION I; RADIATION WORK PROCEDURES AND PRACTICES; RESPIRATORY PROTECTION; PLANT PROCEDURES; PLANT DECONTAMINATION AND HOUSEKEEPING; EQUIPMENT SURVEILLANCE; AND LICENSEE EVENTS. ONE ITEM OF NONCOMPLIANCE WAS IDENTIFIED: LACK OF AN ADEQUATE MECHANISM FOR ISSUANCE OF MANAGEMENT INSTRUCTIONS.

50-219/80-24 - JUN 27: THIS INSPECTION BY ONE REGION-BASED INSPECTOR (8 HRS) WAS LIMITED TO THE EMERGENCY PLANNING AREAS ADDRESSED IN IMMEDIATE ACTION LETTER (IAL 80-13), DATED MAY 16, 1980. NO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED.

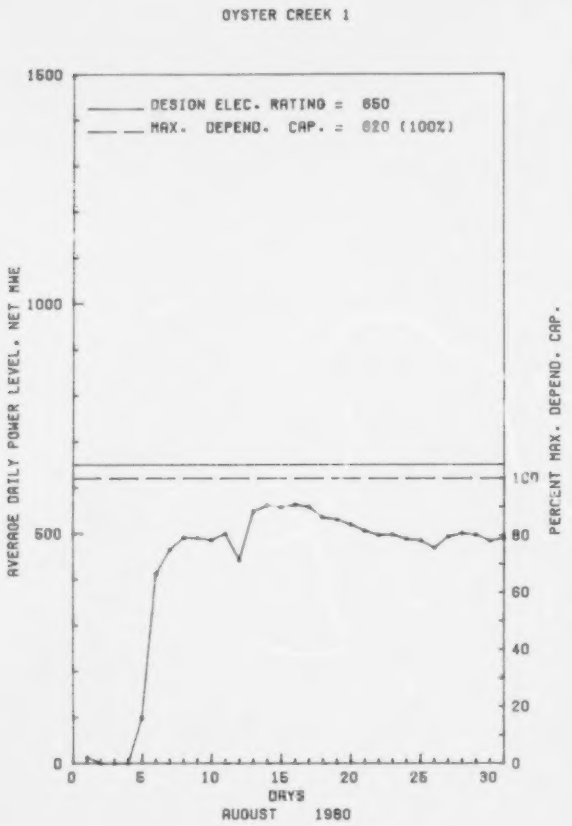
50-219/80-25 - JUL 9 - AUG 1: ROUTINE INSPECTION BY TWO RESIDENT INSPECTORS (139 HRS) OF: FOLLOWUP OF OPERATIONAL EVENTS THAT OCCURRED DURING THE INSPECTION; REVIEW OF PLANT OPERATIONS (STARTUP); TOURS OF THE FACILITY; LOG AND RECORD REVIEWS; AND FOLLOWUP OF IE BULLETINS AND CIRCULARS. NO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED.

REVIEWED BY OMPA LOEB: John R Crooks **DATE:** 10/6/80

REVIEWED BY NRR DL: Walter A. Paulson **DATE:** 10/8/80

OYSTER CREEK

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-219
2. Utility Contact: C. M. McClain (201) 455-8748
3. Licensed Thermal Power (MWt): 1930
4. Nameplate Rating (Gross MWe): 687.5 x 0.8 = 550
5. Design Electrical Rating (Net MWe): 650
6. Maximum Dependable Capacity (Gross MWe): 650
7. Maximum Dependable Capacity (Net MWe): 620
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): 550
10. Reasons for Restrictions, If Any:
CONDENSATE DEMINERALIZER DIFFERENTIAL PRESSURE

ITEM 21 CALCULATED BY USING A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	5,855.0	93,719.0
12. Hours Reactor Was Critical	648.0	1,117.4	69,756.7
13. Reactor Reserve Shutdown Hours	.0	.0	468.2
14. Hours Generator On-Line	639.0	1,035.2	68,340.7
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,051,656	1,605,931	115,485,089
17. Gross Electrical Energy (MWH)	317,890	490,580	39,372,895
18. Net Electrical Energy (MWH)	315,837	474,999	37,956,012
19. Unit Service Factor	85.9	17.7	72.9
20. Unit Availability Factor	85.9	17.7	72.9
21. Unit Capacity Factor (MDC Net)	68.5	13.1	66.9
22. Unit Capacity Factor (DER Net)	65.3	12.5	62.3
23. Unit Forced Outage Rate	14.1	9.2	6.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
3	08/01/80	F	105.0	B	1		ZZ	ZZZZZZ	PLANT MAINTENANCE.

SUMMARY

OYSTER CREEK 1 OPERATED WITH ONE OUTAGE BEGINNING ON THE 1ST FOR PLANT MAINTENANCE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

ENFORCEMENT STATUS

10 CFR 19.12, "INSTRUCTIONS TO WORKERS," STATES, "ALL INDIVIDUALS WORKING IN OR FREQUENTING ANY PORTION OF A RESTRICTED AREA SHALL BE KEPT INFORMED OF THE STORAGE, TRANSFER OR USE OF RADIOACTIVE MATERIALS OR OF RADIATION IN SUCH PORTIONS OF THE RESTRICTED AREA, SHALL BE INSTRUCTED IN THE HEALTH PROTECTION PROBLEMS ASSOCIATED WITH EXPOSURE TO SUCH RADIOACTIVE OR RADIATION, IN PRECAUTIONS OR PROCEDURES TO MINIMIZE EXPOSURE AND IN THE PURPOSE AND FUNCTIONS OF PROTECTIVE DEVICES EMPLOYED." THE EXTENT OF THESE INSTRUCTIONS SHALL BE COMMENSURATE WITH POTENTIAL RADIOLOGICAL HEALTH PROTECTION PROBLEMS IN THE RESTRICTED AREA. CONTRARY TO THE ABOVE, ON MARCH 2, 1980, TWO WORKERS, PERFORMING HANDLING, CLEANING, EXAMINATION AND REPAIR OF HIGHLY CONTAMINATED CONTROL ROD BLADE HANDLING TOOLS WERE NOT GIVEN INSTRUCTIONS COMMENSURATE WITH THE POTENTIAL RADIOLOGICAL HEALTH PROTECTION PROBLEMS ASSOCIATED WITH HANDLING THE TOOLS INCLUDING BUT NOT LIMITED TO PREPLANNING MEETINGS TO DISCUSS RADIOLOGICAL PRECAUTIONS AND PROCEDURES FOR HANDLING THE CONTAMINATED TOOLS. CONSEQUENTLY, THE WORKERS RECEIVED PERSONNEL SKIN CONTAMINATION UP TO 20,000 DPM OVER PORTIONS OF THEIR BODY AND INTAKES OF RADIOACTIVE MATERIAL UP TO 1,600 NANOCURIES OB COBALT-60 FOR THE ONE WORKER AND 500 NANOCURIES COBALT-60 FOR THE OTHER WORKER. THIS REPRESENTS A MAXIMUM INTAKE OF 22% OF THE ALLOWABLE QUARTERLY QUANTITY LIMIT AS SPECIFIED IN 10 CPR 20 FOR COBALT-60.
(8011 INFRACTION)

TECHNICAL SPECIFICATION 6.8.1 STATES IN PART "...WRITTEN PROCEDURES SHALL BE ESTABLISHED, IMPLEMENTED AND MAINTAINED THAT MEET OR EXCEED THE REQUIREMENTS OF SECTION 5.1 AND 5.3 OF AMERICAN NATIONAL STANDARD N18.7-1972...". ANSI N18.7-1972, SECTION 5.3 STATES, IN PART: "NUCLEAR POWER PLANTS SHALL BE OPERATED IN ACCORDANCE WITH WRITTEN PROCEDURES. .EACH PROCEDURE SHALL BE SUFFICIENTLY DETAILED FOR A QUALIFIED INDIVIDUAL TO PERFORM THE REQUIRED FUNCTION WITHOUT DIRECT SUPERVISION...." CONTRARY TO THE ABOVE, INADEQUACIES IN PROCEDURE 205.7, "FUEL CELL ONLOADING/LOADING" PERMITTED THE INTERLOCKS, WHICH PREVENT WITHDRAWAL OF MORE THAN ONE CONTROL ROD WHILE IN "REFUEL" MODE, TO BE IMPROPERLY BYPASSED ON CONTROL RODS 10-23 AND 14-15.
(8019 INFRACTION)

TECHNICAL SPECIFICATION 6.8.1 STATES IN PART "WRITTEN PROCEDURES SHALL BE ESTABLISHED...." PROCEDURE 501 "ANNUNCIATORS AND ALARMS" SECTION 3FG, "FUEL POOL HIGH LEVEL OR HIGH TEMPERATURE" STATES IN PART "ADJUST FUEL POOL LEVEL AS NECESSARY BY DRAINING SYSTEM TO EITHER THE HOTWELL...OR RADWASTE...." CONTRARY TO THE ABOVE, ON MAY 8, 1980, AT APPROXIMATELY 12 NOON A FUEL POOL HIGH LEVEL ALARM WAS ANNUNCIATED ON PANEL 3FG. NO ATTEMPT WAS MADE TO LOWER THE LEVEL IN ACCORDANCE WITH PROCEDURE 501. SUBSEQUENTLY AT ABOUT 6:00 PM APPROXIMATELY 1000 GALLONS OF RADIOACTIVE FUEL POOL WATER OVERFLOWED INTO THE REACTOR BUILDING VENTILATION SYSTEM.
(8019 INFRACTION)

TECHNICAL SPECIFICATION 6.8.1 STATES IN PART, "WRITTEN PROCEDURES SHALL BE ESTABLISHED, IMPLEMENTED AND MAINTAINED THAT MEET OR EXCEED THE REQUIREMENTS OF SECTIONS 5.1 AND 5.3 OF AMERICAN NATIONAL STANDARD N18.7-1972. ..." AMERICAN NATIONAL STANDARD N18.7-1972, SECTION 5.1.4, "SPECIAL ORDERS", STATES IN PART, "A MECHANISM SHALL BE PROVIDED FOR ISSUING MANAGEMENT INSTRUCTIONS THAT HAVE SHORT TERM APPLICABILITY AND REQUIRE DISSEMINATION. SUCH INSTRUCTIONS, SOMETIMES REFERRED TO AS SPECIAL ORDERS, SHOULD ENCOMPASS SPECIAL OPERATIONS OR OTHER SIMILAR MATTERS...." CONTRARY TO THE ABOVE, THERE IS NO ADEQUATE MECHANISM PROVIDED FOR ISSUANCE OF MANAGEMENT INSTRUCTIONS OF SHORT TERM APPLICABILITY THAT REQUIRE DISSEMINATION. THIS LACK OF MANAGEMENT AND PROCEDURAL CONTROLS RESULTED IN ABOUT A 16 HOUR DELAY ON JUNE 5 1980 IN ESTABLISHING A FIRE WATCH PATROL, REQUIRED BY TECHNICAL SPECIFICATION 3.12.E.2, TO BE INITIATED WITHIN ONE HOUR WHEN CERTAIN FIRE PROTECTION BARRIERS WERE FOUND TO BE NON-FUNCTIONAL, AFTER VERBAL INSTRUCTIONS TO INITIATE THE FIRE WATCH WERE GIVEN BY THE STATION SUPERINTENDENT.
(8025 INFRACTION)

INSPECTION STATUS

PALISADES

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1. Location (state/county) (nearest population center)	Michigan/Van Buren 5 miles S of South Haven
2. Electric Reliability Council	East Central Area Reliability Coordination Agreement
3. Type of Reactor	PWR
4. Power Level (MWT)	2530
5. Design Electrical Rating (MWE Net)	805
6. Date of Initial Criticality	May 24, 1971
7. Date of Commercial Operation	December 31, 1971
8. Condenser Cooling Method	Forced Draft Cooling Towers
9. Condenser Cooling Water Source	Lake Michigan
Utility & Contractor Information	
10. Licensee	Consumers Power Company
11. Corporate Address	212 W. Michigan Avenue Jackson, Michigan 49201
12. Corporate Contact	R. B. DeWitt
13. Architect/Engineer	Bechtel
14. Nuclear Steam Supply System	Combustion Engineering
15. Constructor	Bechtel
Regulatory Information	
16. IE Region responsible	III
17. IE Branch Chief	R. Heishman
18. IE Section Chief	D.C. Boyd
19. IE Resident Inspector	B. Jorgenson (616) 764-8971
20. Licensing Project Manager	R. D. Silver
21. Docket Number	50-255
22. License Number/Date of Issue	DPR-20, 3/24/71
23. Public Document Room	Kalamazoo Public Library 315 South Rose Street Reference Department Kalamazoo, Michigan 49006

INSPECTION STATUS

ENFORCEMENT STATUS

PALISADES TECHNICAL SPECIFICATIONS SECTION 6.10.1E REQUIRES RECORDS OF TRAINING AND QUALIFICATION FOR MEMBERS OF PLANT STAFF TO BE RETAINED FOR FIVE YEARS. ADMINISTRATIVE PROCEDURE 13 DELINEATES SPECIFIC TRAINING REQUIREMENTS AND RECORDS. CONTRARY TO THE ABOVE, RECORDS OF TRAINING FOR SEVERAL EMPLOYEES DID NOT EXIST IN THE FOLLOWING AREAS: (A) CRANE SAFETY TRAINING, FORKLIFT TRAINING, AND CONTROLLED MATERIAL PROCEDURES TRAINING (EVERY 2 YEARS PER ADMINISTRATIVE PROCEDURE 13.3); AND (B) INDUSTRIAL SAFETY AND QUALITY ASSURANCE INDOCTRINATION (ADMINISTRATIVE PROCEDURE 13.1).
(8009 DEFICIENCY)

IE BULLETIN 79-05C "NUCLEAR INCIDENT AT THREE MILE ISLAND - SUPPLEMENT" AND THE LICENSEE'S RESPONSE LETTER DATED AUGUST 2, 1979, ESTABLISHED THE LICENSEE'S FORMAL COMMITMENT TO PROVIDE TWO LICENSED OPERATORS IN THE CONTROL ROOM AT ALL TIMES DURING PLANT OPERATION. CONTRARY TO THE ABOVE, AT APPROXIMATELY NOON ON JUNE 30, 1980, WITH THE PLANT IN OPERATION, THE CONTROL ROOM WAS BRIEFLY OBSERVED TO BE ATTENDED BY ONLY A SINGLE LICENSED OPERATOR.
(8010 DEVIATION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENTS

A reactor trip from significant power level was manually initiated on July 2 when severe oil leakage developed on the turbine seal oil system. A reactor trip in the startup range was experienced on July 9 due to an erroneous high startup rate signal from a defective wide-range nuclear instrument channel. (80-13)

MANAGERIAL ITEMS

Effective August 1, 1980, Robert W. Montross assumed duties as Plant General Manager - Palisades. He replaces James G. Lewis, who transferred to the licensee's corporate Nuclear Operations staff effective the same date. (80-13)

PLANT STATUS

Plant taken to cold shutdown mode in August to change out several leaking CRD seals. Tentative restart date is September 2, 1980.

LAST IE SITE INSPECTION DATE: July 1-31, 1980

INSPECTION REPORT NO: 80-13

SUMMARY

INSPECTION SUMMARY

Inspection on July 22-24, (80-11): Routine, unannounced inspection of core thermal power evaluation; power coefficient of reactivity measurement; control rod worth measurements; quadrant power tilt measurements; and zero power ARO (all rods out) and critical boron concentration. The inspection involved 36 inspector-hours onsite by three NRC inspectors. No items of noncompliance or deviations were identified.

Inspection during July, (80-13): Routine, monthly resident inspection program activities including operational safety; maintenance; surveillance; reportable events; plant trips; organization and administration; review and audit; unresolved items; and action on NRC correspondence. The inspection involved 240 inspector-hours onsite by two NRC inspectors, including 44 inspector-hours onsite during off-shifts. Of the nine areas inspected, no items of noncompliance or deviations were identified in eight areas. One item of noncompliance (failure to adhere to administrative procedures for equipment control) was identified in the remaining area.

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-20/ 01T-0	07/11/80	07/29/80	CRDM No. 39 drove in approximately 39 inches.
80-21/ 01T-0	07/25/80	08/06/80	An operator inadvertently opened containment sump valve CV-3030.
80-22/ 03L-0	07/10/80	08/08/80	Two hours after reactor criticality NI-03 had not been declared operable prior to reactor startup.
80-24/ 01T-0	07/31/80	08/12/80	With D/G 1-1 inoperable to perform a mod. and charging pump P-55A inoperable for repairs, P-55B became inoperable.
80-25/ 01T-0	08/01/80	08/13/80	An unplanned gaseous release occurred w/o the 15 day minimum holdup required by TS.

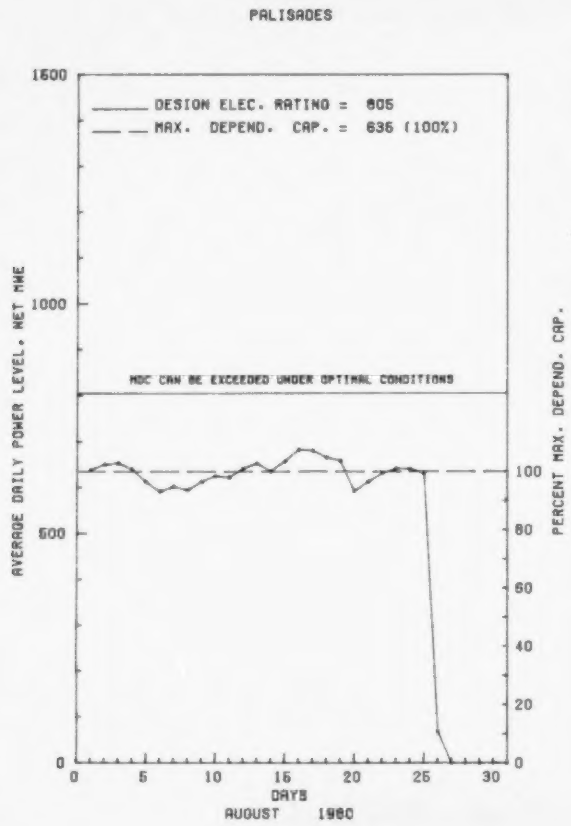
==

REVIEWED BY OMPA LOEB: MR Beebe **DATE:** 10/06/80

REVIEWED BY NRR DL: J. V. Wambach **DATE:** 10/08/80

PALISADES

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-255
2. Utility Contact: P. J. Bodtke (616) 764-8913
3. Licensed Thermal Power (MWt): 2530
4. Nameplate Rating (Gross MWe): 955 x 0.85 = 812
5. Design Electrical Rating (Net MWe): 805
6. Maximum Dependable Capacity (Gross MWe): 675
7. Maximum Dependable Capacity (Net MWe): 635
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	5,855.0	76,286.0
12. Hours Reactor Was Critical	603.7	2,061.7	41,626.5
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	603.7	1,965.8	39,392.3
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,335,552	4,279,848	75,791,328
17. Gross Electrical Energy (MWH)	406,390	1,316,990	23,349,220
18. Net Electrical Energy (MWH)	382,051	1,238,114	21,908,889
19. Unit Service Factor	81.1	33.6	51.6
20. Unit Availability Factor	81.1	33.6	51.6
21. Unit Capacity Factor (MDC Net)	80.9	33.3	45.2
22. Unit Capacity Factor (DER Net)	63.8	26.3	35.7
23. Unit Forced Outage Rate	18.9	14.9	35.4

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
FIRE PROTECT. MODS - 11/80; TMI MODS - 1/81.
25. If Shut Down At End of Report Period, Estimated Date of Startup: 09/01/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
17	08/26/80	F	140.3	A	1				CONDENSATE PUMP TRIP.

SUMMARY

PALISADES OPERATED NORMALLY WITH ONE OUTAGE ON THE 26TH FOR A CONDENSATE PUMP TRIP.

LEGEND

[1] Type:
F-Forced
S-Scheduled

[2] Reason:
A-Equipment Failure
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error
H-Other

[3] Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Continuations
5-Load Reduction
9-Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

PEACH BOTTOM 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Pennsylvania/York 19 miles S of Lancaster
2.	Electric Reliability Council	Mid-Atlantic Area Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	3293
5.	Design Electrical Rating (MWE Net)	1065
6.	Date of Initial Criticality	September 16, 1973
7.	Date of Commercial Operation	July 5, 1974
8.	Condenser Cooling Method	Once Through with Cooling Towers
9.	Condenser Cooling Water Source	Conowingo Pond/Susquehanna River

Utility & Contractor Information

10.	Licensee	Philadelphia Electric Company
11.	Corporate Address	2301 Market Street Philadelphia, Pennsylvania 19101
12.	Corporate Contact	S. L. Daltroff, Vice President Electric Production
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	GE
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	IE Section Chief	E. McCabe
19.	IE Resident Inspector	C. Cowgill 717-456-7614
20.	Licensing Project Manager	D. Verrelli
21.	Docket Number	50-277
22.	License Number/Date of Issue	DPR-44, 8/8/73
23.	Public Document Room	Government Publications Section State Library of Pennsylvania Education Building Commonwealth and Walnut Street Harrisburg, Pennsylvania 17126

INSPECTION STATUS

ENFORCEMENT STATUS

TECHNICAL SPECIFICATION 6.11 REQUIRES THAT PROCEDURES FOR PERSONNEL RADIATION PROTECTION SHALL BE ADHERED TO. HPO/CO-10A "CONDUCT IN CONTROLLED AREAS STATES IN PART 1. ALWAYS WEAR PERSONNEL DOSIMETRY... 2. SIGNS ARE POSTED AT VARIOUS AREAS TO ALERT PERSONNEL TO THE RADIATION CONDITIONS..." CONTRARY TO THE ABOVE, ON JANUARY 7, 1980, ONE OF TWO INDIVIDUALS WORKING IN THE UNIT 2 HOT TOOL ROOM, A CONTROLLED AREA IN THE UNIT 2 REACTOR BUILDING, WAS FOUND NOT WEARING HIS PROVIDED PERSONNEL DOSIMETRY. FURTHER THIS AREA WAS POSTED AS REQUIRING A RWP AND NO RWP WAS IN EFFECT.
(8001 INFRACTION)

FAILURE TO HAVE A HIGH RADIATION DOOR LOCKED.
(8015 INFRACTION)

FAILURE TO HAVE A DOUBLE BARRIER AROUND A VITAL AREA.
(8015 INFRACTION)

PICTURE BADGE ISSUED TO A CONTRACTOR FOR ANOTHER CONTRACTOR.
(8015 INFRACTION)

INADEQUATE LIGHTING IN THE PROTECTED AREA.
(8015 INFRACTION)

METAL DETECTOR BEING USED FOR SEARCH WAS INOPERABLE.
(8015 INFRACTION)

FAILURE TO CONDUCT DRILLS IDENTIFIED IN THE PHYSICAL SECURITY PLAN
(8015 INFRACTION)

FAILURE TO CHANGE SECURITY LOCKS WHEN AN EMPLOYEE WHO HAD ACCESS TO THE KEYS TERMINATED.
(8015 INFRACTION)

BASED ON THE RESULTS OF AN NRC INSPECTION CONDUCTED ON JUNE 5, 1980, IT APPEARS THAT ONE ACTIVITY WAS NOT CONDUCTED IN FULL COMPLIANCE WITH NRC REQUIREMENTS AS INDICATED BELOW. THIS IS A DEFICIENCY. 10 CFR 71.5(A), TRANSPORTATION OF LICENSED MATERIAL, REQUIRES THAT NRC LICENSEES COMPLY WITH THE APPLICABLE PACKAGING AND TRANSPORTATION REQUIREMENTS OF THE DEPARTMENT OF TRANSPORTATION IN 49 CFR PARTS 170-189. 49 CFR 172.504(A), GENERAL PLACARDING REQUIREMENTS, REQUIRES THAT EACH MOTOR VEHICLE CONTAINING ANY QUANTITY OF HAZARDOUS MATERIAL MUST BE PLACARDED ON EACH END AND EACH SIDE WITH THE SPECIFIED PLACARD. CONTRARY TO THE ABOVE, ON JUNE 5, 1980, THE LICENSEE DELIVERED LOW LEVEL RADIOACTIVE MATERIAL VIA TRI-STATE MOTOR TRANSPORT COMPANY TRAILER #SL-1 (SSR #020598, VOLUME ALLOCATION 30680-311-E) TO THE CHEM NUCLEAR BURIAL SITE WITHOUT A "RADIOACTIVE" PLACARD ON THE FRONT OF EITHER THE TRACTOR (#633) OR THE TRAILER.
(8027 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

MANAGERIAL ITEMS:

THE STATION MAINTENANCE ENGINEER, MR. T. S. SPENCER, WAS TRANSFERRED TO THE LIMERICK SITE; MR. J. K. DAVENPORT WAS NAMED AS HIS REPLACEMENT EFFECTIVE AUGUST 18.

PLANT STATUS:

FOLLOWING A 4 1/2 MONTH OUTAGE FOR REFUELING AND TORUS MODIFICATIONS, THE UNIT WAS STARTED UP ON 8/9, SCRAMMED PER IE BULLETIN 80-17 ON 8/13 AND 8/14, AND PLACED ON LINE 8/17. THE REACTOR OPERATED AT A NOMINAL 100 PERCENT POWER FOR THE REMAINDER OF THE MONTH. REPRESENTATIVE OFFGAS RATE WAS 2700 MICROCURIES PER SECOND: REPRESENTATIVE COMBINED UNIT 2 AND UNIT 3 STACK GAS RELEASE RATE WAS 170 MICROCURIES PER SECOND.

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-011/ 01T	07/14/80	07/28/80	DIESEL GENERATOR CARDOX SYSTEM INOPERABLE DUE TO MISPOSITIONED VALVE. VALVE REOPENED AND SYSTEM RESTORED.
80-012/ 01T	07/21/80	08/04/80	BACKUP SCRAM SOLENOID VALVES INOPERABLE DUE TO IMPROPER VOLTAGE RATING. SOLENOID INSTALLED 250 VOLT SOLENOID POWER SUPPLY 125 VOLT.

LAST IE SITE INSPECTION DATE: 8/1-31/80

INSPECTION REPORT NO: 50-277/80-28

SUMMARY

INSPECTION SUMMARY

50-277/80-23 - JUL 1-3: ROUTINE, ANNOUNCED INSPECTION BY ONE REGION-BASED INSPECTOR (24 HRS) OF DOCUMENTATION ON ENVIRONMENTAL QUALIFICATION OF CLASS 1E ELECTRICAL EQUIPMENT INSIDE CONTAINMENT ASSOCIATED WITH THE MAIN STEAM AND FEEDWATER SYSTEM, ELECTRICAL POWER SYSTEM AND THE RHR SYSTEM AS DEFINED IN THE LICENSEE RESPONSE OF APRIL 15, 1980 TO IE BULLETIN 79-01B. NO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED.

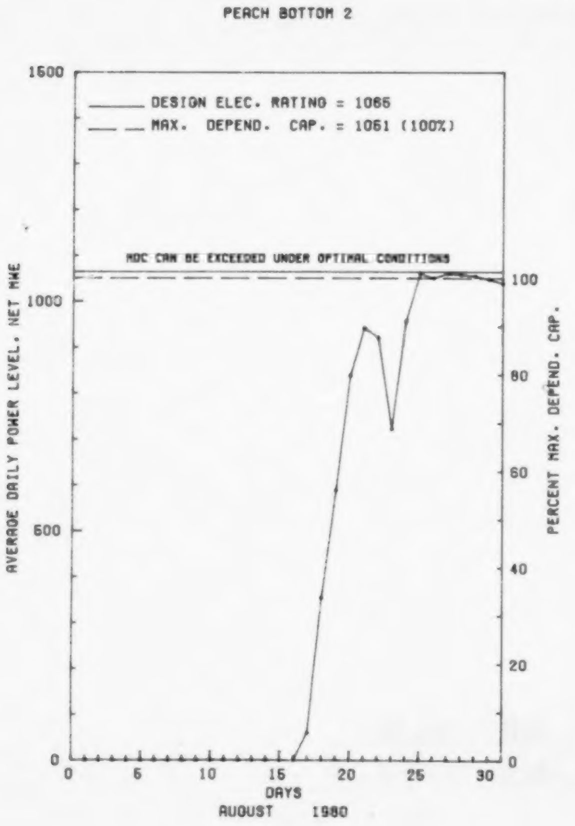
50-277/80-27 - JUN 5: VISIT TO CHEM-NUCLEAR WASTE DISPOSAL SITE, BARNWELL, SOUTH CAROLINA FOR UNANNOUNCED INSPECTIONS OF LICENSEE-SHIPPERS. EACH LICENSEE-SHIPPER VEHICLE WAS INSPECTED FOR COMPLIANCE WITH DEPARTMENT OF TRANSPORTATION (DOT) AND NUCLEAR REGULATORY COMMISSION (NRC) REGULATIONS AS FOLLOWS: (1) SHIPPING PAPER REQUIREMENTS; (2) DOT PLACARDING REQUIREMENTS; (3) RADIATION LEVELS; (4) REMOVABLE CONTAMINATION; (5) DOT MARKING AND LABELING REQUIREMENTS FOR PACKAGES; (6) DOT AND NRC REQUIREMENTS FOR EXTERNAL PACKAGE FEATURES: AND (7) PROHIBITED ARTICLES OR CONTENTS. ONE ITEM OF NONCOMPLIANCE WAS IDENTIFIED: FAILURE TO PLACARD FRONT OF VEHICLE WITH "RADIOACTIVE" SIGN.

REVIEWED BY OMPA LOEB: John R Crooks **DATE:** 10/6/80

REVIEWED BY NRR DL: **DATE:** 10-8-80

PEACH BOTTOM 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-277
2. Utility Contact: W. M. Alden (215) 841-5022
3. Licensed Thermal Power (MWt): 3293
4. Nameplate Rating (Gross MWe): 1280 x 0.9 = 1152
5. Design Electrical Rating (Net MWe): 1065
6. Maximum Dependable Capacity (Gross MWe): 1098
7. Maximum Dependable Capacity (Net MWe): 1051
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	5,855.0	53,999.0
12. Hours Reactor Was Critical	518.0	1,959.0	35,714.0
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	351.1	1,741.3	38,724.1
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	968,748	4,800,518	111,466,162
17. Gross Electrical Energy (MWH)	319,400	1,575,380	36,633,410
18. Net Electrical Energy (MWH)	300,961	1,475,207	35,290,992
19. Unit Service Factor	47.2	29.7	71.7
20. Unit Availability Factor	47.2	29.7	71.7
21. Unit Capacity Factor (MDC Net)	38.5	24.0	61.8
22. Unit Capacity Factor (DER Net)	38.0	23.7	61.0
23. Unit Forced Outage Rate	11.2	3.1	6.2

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
9	03/21/80	S	301.8	C	4		RC	FUELXX	CONTINUATION OF REFUEL OUTAGE.
9	08/13/80	S	25.2	D	9		RA	CRDRVE	MANUAL SCRAM IN ACCORDANCE WITH NRC I.E. BULLETIN 80-17.
9	08/14/80	S	21.8	D	9		RA	CRDRVE	AUTOMATIC SCRAM IN ACCORDANCE WITH NRC I.E. BULLETIN 80-17.
10	08/15/80	F	44.1	A	9		HA	TURBIN	RECOMBINER LOOP SEAL MODIFICATION.

SUMMARY

PEACH BOTTOM 2 ENDED ITS REFUELING OUTAGE AND REQUIRED NRC TESTING ON THE 17TH, AND BROUGHT POWER GENERATION TO FULL LOAD BY THE END OF THE MONTH.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

PEACH BOTTOM 3

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Pennsylvania/York 19 miles S of Lancaster
2.	Electric Reliability Council	Mid-Atlantic Area Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	3293
5.	Design Electrical Rating (MWE Net)	1065
6.	Date of Initial Criticality	August 7, 1974
7.	Date of Commercial Operation	December 23, 1974
8.	Condenser Cooling Method	Once Through with Cooling Towers
9.	Condenser Cooling Water Source	Conowingo Pond/Susquehanna River

Utility & Contractor Information

10.	Licensee	Philadelphia Electric Company
11.	Corporate Address	2301 Market Street Philadelphia, Pennsylvania 19101
12.	Corporate Contact	S. L. Daltroff, Vice President Electric Production
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	GE
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	IE Section Chief	E. McCabe
19.	IE Resident Inspector	C. Cowgill (717-456-7614)
20.	Licensing Project Manager	D. Verrelli
21.	Docket Number	50-278
22.	License Number/Date of Issue	DPR-56, 7/2/74
23.	Public Document Room	Government Publications Section State Library of Pennsylvania Education Building Commonwealth and Walnut Street Harrisburg, Pennsylvania 17126

INSPECTION STATUS

ENFORCEMENT STATUS

FAILURE TO HAVE HIGH RADIATION DOOR LOCKED.
(8013 INFRACTION)

FAILURE TO HAVE A DOUBLE BARRIER AROUND A VITAL AREA.
(8013 INFRACTION)

PICTURE BADGE ISSUED TO A CONTRACTOR FOR ANOTHER CONTRACTOR.
(8013 INFRACTION)

INADEQUATE LIGHTING IN THE PROTECTED AREA.
(8013 INFRACTION)

METAL DETECTOR BEING USED FOR SEARCH WAS INOPERABLE.
(8013 INFRACTION)

FAILURE TO CONDUCT DRILLS IDENTIFIED IN THE PHYSICAL SECURITY PLAN.
(8013 INFRACTION)

FAILURE TO CHANGE SECURITY LOCKS WHEN AN EMPLOYEE WHO HAD ACCESS TO THE KEYS TERMINATED.
(8013 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

MANAGERIAL ITEMS:

STATION MAINTENANCE ENGINEER, MR. T. S. SPENCER, WAS TRANSFERRED TO THE LIMERICK SITE; MR. J. K. DAVENPORT WAS NAMED AS HIS REPLACEMENT, EFFECTIVE AUGUST 18.

PLANT STATUS:

THE UNIT OPERATED AT A NORMAL 100 PERCENT POWER. DURING THE MONTH A SCRAM OCCURRED ON 8/4 DURING ROUTINE TURBINE VALVE TESTING DUE TO AN EHC SYSTEM PROBLEM. SCRAM TESTS PER IE BULLETIN 80-17 WERE CONDUCTED ON 8/8 AND 8/9. THE UNIT WAS SHUTDOWN FOR FIVE DAYS COMMENCING 8/15 FOR REPAIR OF THE RHR "A" LOOP LPCI INJECTION VALVE. REPRESENTATIVE OFFGAS RATE WAS 3600 MICROCURIES PER SECOND; REPRESENTATIVE COMBINED UNIT 2 AND UNIT 3 STACK GAS RELEASE RATE WAS 170 MICROCURIES PER SECOND.

LAST IE SITE INSPECTION DATE: 8/1-31/80

INSPECTION REPORT NO: 50-278/80-20

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-013/ 03L	06/05/80	06/30/80	A DRYWELL PRESSURE SWITCH USED TO INITIATE THE REACTOR PROTECTION SYSTEM WAS FOUND OUT OF CALIBRATION DURING SURVEILLANCE TESTING. THE SWITCH WAS RECALIBRATED AND TESTED.
80-014/ 03L	06/08/80	07/14/80	CONTAINMENT ISOLATION VALVE MO3-10-26B FAILED OPEN. REDUNDANT ISOLATION VALVE IS MAINTAINED CLOSED.
80-016/ 01T	07/17/80	07/31/80	ANALYSIS OF AS BUILT CONFIGURATION SHOWED THAT SUPPORT 3-10-CB-S-44 EXCEEDED ALLOWABLE STRESS IN DESIGN BASIS EARTHQUAKE ASSOCIATED PIPING REMAINED OPERABLE WITH INOPERABLE SUPPORT.

SUMMARY

INSPECTION SUMMARY

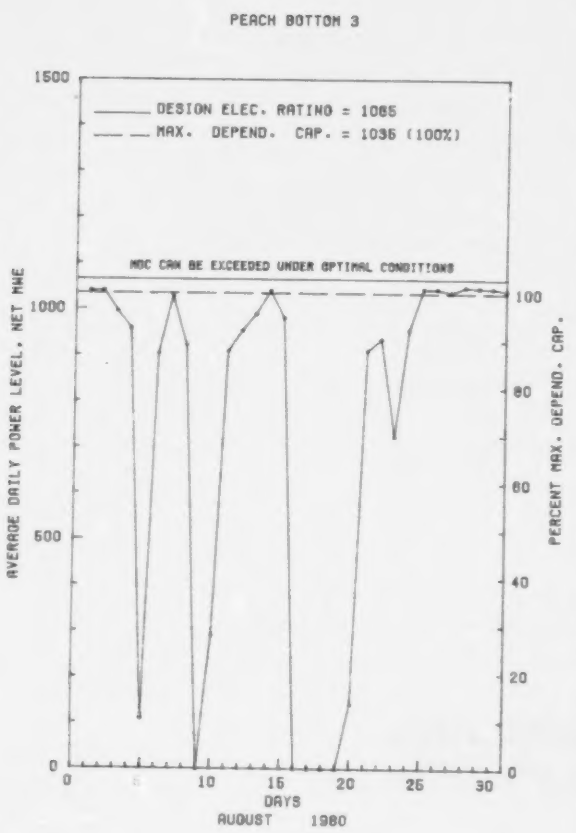
50-278/80-17 - JUL 1-31: ROUTINE, ANNOUNCED INSPECTION BY ONE REGION-BASED INSPECTOR (24 HRS) OF DOCUMENTATION ON ENVIRONMENTAL QUALIFICATION OF CLASS 1E ELECTRICAL EQUIPMENT INSIDE CONTAINMENT ASSOCIATED WITH THE MAIN STEAM AND FEEDWATER SYSTEM, ELECTRICAL POWER SYSTEM AND THE RHR SYSTEM AS DEFINED IN THE LICENSEE RESPONSE OF APRIL 15, 1980 TO IE BULLETIN 79-01B. NO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED.

REVIEWED BY OMPA LOEB: **DATE:** 10/6/80

REVIEWED BY NRR DL: **DATE:** 10-8-80

PEACH BOTTOM 3

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-278
2. Utility Contact: W. M. Alden (215) 841-5022
3. Licensed Thermal Power (MWt): 3293
4. Nameplate Rating (Gross MWe): 1280 x 0.9 = 1152
5. Design Electrical Rating (Net MWe): 1065
6. Maximum Dependable Capacity (Gross MWe): 1098
7. Maximum Dependable Capacity (Net MWe): 1035
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	5,855.0	49,895.1
12. Hours Reactor Was Critical	606.0	4,708.0	39,554.0
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	577.8	4,538.2	38,470.0
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,779,790	14,431,273	108,650,427
17. Gross Electrical Energy (MWH)	578,800	4,803,250	35,389,110
18. Net Electrical Energy (MWH)	553,363	4,625,037	33,969,837
19. Unit Service Factor	77.7	77.5	77.1
20 Unit Availability Factor	77.7	77.5	77.1
21. Unit Capacity Factor (MDC Net)	71.9	76.3	65.8
22. Unit Capacity Factor (DER Net)	69.8	74.2	63.9
23. Unit Forced Outage Rate	4.5	14.1	7.8

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE

25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
13	08/04/80	[illegible]	17.5	B	3		HB	INSTRU	LOW EHC PRESSURE DURING STOP VALVE TESTING CAUSED AUTOMATIC SCRAM.
14	08/08/80	S	15.8	D	2		RA	CRDRVE	MANUAL SCRAM IN ACCORDANCE WITH NRC I.E. BULLETIN 80-17.
14	08/09/80	S	10.2	D	3		RA	CRDRVE	AUTOMATIC SCRAM IN ACCORDANCE WITH NRC I.E. BULLETIN 80-17.
14	08/10/80	F	9.6	H	3		RB	INSTRU	AUTOMATIC SCRAM DUE TO APRM HI-FLUX.
15	08/15/80	S	113.1	B	3	80-19	CF	VALVEX	SHUTDOWN TO REPAIR OPERATING MECHANISMS OF LPCI INJECTION VALVE.

SUMMARY

PEACH BOTTOM 3 OPERATED WITH 5 OUTAGES TOTALING 113.1 HOURS LISTED ABOVE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

PILGRIM

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Massachusetts/Plymouth 4 miles SE of Plymouth
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	BWR
4.	Licenses Power Level (MWT)	1998
5.	Design Electrical Rating (MWE Net)	655
6.	Date of Initial Criticality	June 16, 1972
7.	Date of Commercial Operation	December 1972
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Cape Cod Bay

Utility & Contractor Information

10.	Licensee	Boston Edison Company
11.	Corporate Address	800 Boylston Street Boston, Massachusetts 02199
12.	Corporate Contact	A. Corey, Vice President
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	GE
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	IE Section Chief	T. MARTIN
19.	IE Resident Inspector	J. JOHNSON
20.	Licensing Project Manager	J. Hannon (X7872)
21.	Docket Number	50-293
22.	License Number/Date of Issue	DPR-35, 6/8/72
23.	Public Document Room	Plymouth Public Library North Street Plymouth, Massachusetts 02360

INSPECTION STATUS

ENFORCEMENT STATUS

FAILURE TO IMPLEMENT ACCESS CONTROL PROCEDURE. (8022 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

1. THE HPCI SYSTEM EXHIBITED SPEED CONTROL PROBLEMS AND A TECHNICAL REPRESENTATIVE WAS CALLED IN TO EVALUATE. THIS SYSTEM HAS HAD A HIGHER THAN NORMAL FAILURE RATE AND ITS RELIABILITY IS BEING EVALUATED.
2. THE MODE SWITCH IN SHUTDOWN SCRAM WAS FOUND TO BE DEFEATED BY A LOSS AND REENERGIZATION OF ONE RPS BUS. THE CIRCUIT WAS MANUALLY RESET. A DESIGN EVALUATION IS IN PROGRESS.
3. THE "D" CONDENSATE DEMINERALIZER PASSED RESIN. RESULTING IN A PLANT SHUTDOWN DUE TO HIGH REACTOR WATER CONDUCTIVITY.
4. THE APRM GAIN ADJUSTMENTS WERE FOUND TO BE NONCONSERVATIVE CAUSING THE ACTUAL SCRAM SETPOINT TO EXCEED TECHNICAL SPECIFICATION VALUES.

MANAGERIAL ITEMS:

THE CORPORATE AND SITE NUCLEAR OPERATIONS MANAGEMENT ORGANIZATION IS UNDERGOING A MAJOR CHANGE.

PLANT STATUS:

THE PLANT OPERATED AT 100% POWER FOR MOST OF THE MONTH WITH OFFGAS READINGS OF APPROXIMATELY 34,000 UCI/SEC AND A MAIN STACK RELEASE RATE OF 3,373 UCI/SEC.

LAST IE SITE INSPECTION DATE: 8/1-31/80

INSPECTION REPORT NO: 50-293/80-26

SUMMARY

INSPECTION SUMMARY

50-293/80-23 - MAY 28-29: INSPECTION BY ONE REGION-BASED INSPECTOR (18 HRS) OF EMERGENCY PLANNING AREAS ADDRESSED IN IMMEDIATE ACTION LETTER (IAL 80-06) DATED FEBRUARY 27, 1980. NO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

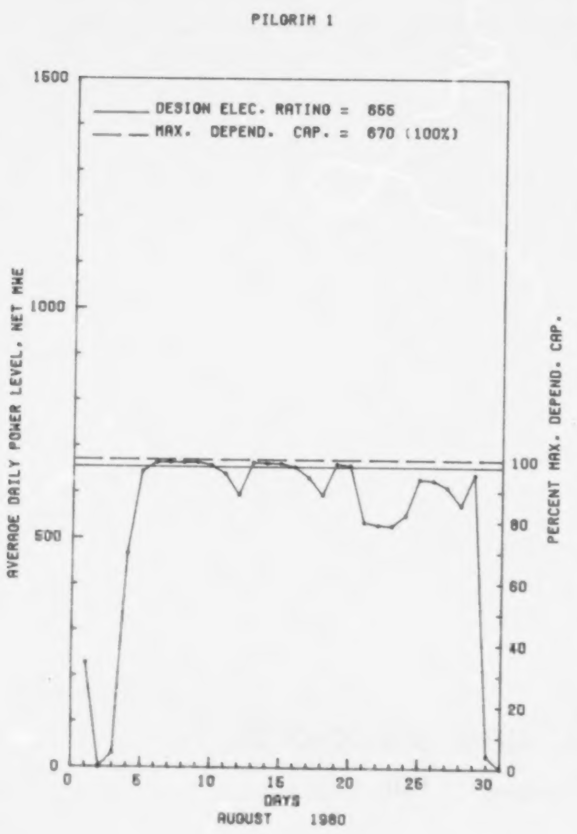
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE			
80-016/ 03L	05/13/80	05/23/80	CORE SPRAY PUMP B AND CORE SPRAY VALVE 24B FAILED TO OPERATE AUTOMATICALLY DURING SURVEILLANCE TEST.
80-018/ 03L	05/23/80	06/12/80	OXYGEN ANALYZER VALVES INOPERABLE.
80-019/ 03L	05/19/80	06/13/80	HPCI AND RCIC CONTROL PROBLEMS DURING S/U TESTING FOLLOWING REFUELING.
80-020/ 03L	04/26/80	06/18/80	APP J TYPE C TEST REPORT. PCIVS FAILED SEAT LEAKAGE DURING REFUELING OUTAGE.
80-021/ 04T	05/08/80	06/25/80	ANOMOLOUS MEASUREMENT OF COBALT-60 IN IRISH MOSS FROM D/C CANAL
80-022/ 03L	05/27/80	06/25/80	SETPOINT DRIFT-RCIC STEAM LINE HI TEMP SWITCH.
80-023/ 03L	05/26/80	06/26/80	ROD BLOCK MONITOR INOPERABLE.
80-024/ 03L	05/30/80	07/01/80	RCIC SYSTEM INOPERABLE.
80-025/ 03L	06/02/80	07/01/80	FIRE DETECTION ALARM FROM HPCI QUADRANT DUE TO VALVE LEAK.
80-026/ 03L	06/03/80	07/02/80	FAILURE TO REPORT REACTOR SCRAM.
80-027/ 03L	06/05/80	07/02/80	MAIN STACK GAS MONITOR PUMP FAILURE.
80-028/ 03L	06/21/80	08/05/80	OFF GAS SAMPLE FLOW LOW.
80-029/ 01T	07/15/80	07/29/80	CONTAINMENT ATMOSPHERE RADIATION MONITOR C19 INOPERABLE.
80-030/ 03L	07/25/80	07/29/80	D SRV FAILED TO OPEN.
80-031/ 04T	07/21/80	07/30/80	ANOMOLOUS SAMPLE OF COBALT-60 IN MUSSELS FROM D/C CANAL.
80-033/ 03L	07/01/80	07/30/90	NITROGEN PURGE VALVE FAILED CLOSING TIME.
80-034/ 03L	07/01/80	07/31/80	HPCI EXHAUST LINE SNUBBER FAILURES.
80-035/ 03L	07/07/80	08/06/80	INOP FIRE DETECTORS IN AOG BLDG.
80-036/ 01T	08/06/80	08/14/80	HOURLY FIRE WATCH NOT PERFORMED FOR INOP SMOKE DETECTORS.
80-037/ 03L	07/09/80	08/08/80	SALT SERVICE WATER PUMP THERMAL OVERLOAD TRIP (P208D) WITHOUT PUMP OR MOTOR MALFUNCTION.

REVIEWED BY OMPA LOEB: John L Crooks **DATE:** 10/6/80

REVIEWED BY NRR DL: John N. Hannon **DATE:** 10/8/80

PILGRIM

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-293
2. Utility Contact: G. G. Whitney (617) 746-7900
3. Licensed Thermal Power (MWt): 1998
4. Nameplate Rating (Gross MWe): 780 x 0.87 = 678
5. Design Electrical Rating (Net MWe): 655
6. Maximum Dependable Capacity (Gross MWe): 690
7. Maximum Dependable Capacity (Net MWe): 670
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	5,855.0	67,751.0
12. Hours Reactor Was Critical	668.3	2,536.8	47,517.7
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	640.2	2,397.1	45,949.6
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,202,352	4,260,552	77,351,664
17. Gross Electrical Energy (MWH)	408,230	1,449,290	25,616,124
18. Net Electrical Energy (MWH)	392,436	1,393,419	24,599,542
19. Unit Service Factor	86.0	40.9	67.8
20. Unit Availability Factor	86.0	40.9	67.8
21. Unit Capacity Factor (MDC Net)	78.7	35.5	54.2
22. Unit Capacity Factor (DER Net)	80.5	36.3	55.4
23. Unit Forced Outage Rate	8.2	4.1	10.3

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
6	08/01/80	F	57.4	A	2	80-043	HG	DEMINX	HIGH CONDUCTIVITY IN REACTOR WATER. LATERAL IN DEMINERALIZER LOOSE. PLUGGED AND RETURNED TO SERVICE.
7	08/30/80	S	46.4	A	2		CH	HTEXCH	SCHEDULED PLANT OUTAGE FOR 4TH POINT HEATER TUBE LEAK REPAIR

SUMMARY

PILGRIM OPERATED WITH TWO OUTAGES DURING THE MONTH. OUTAGE NO. 6 WAS DUE TO A LOOSE LATERAL IN THE DEMINERALIZER, WHILE OUTAGE NO. 7 WAS DUE TO A HEATER TUBE LEAK.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H. Same source.

POINT BEACH 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Wisconsin/Manitowoc 15 miles N of Manitowoc
2.	Electric Reliability Council	Mid-America Interpool Network
3.	Type of Reactor	PWR
4.	Power Level (MWT)	1518
5.	Design Electrical Rating (MWE Net)	497
6.	Date of Initial Criticality	November 2, 1970
7.	Date of Commercial Operation	December 21, 1970
8.	Condenser Cooling Method	Straight Through
9.	Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10.	Licensee	Wisconsin Electric Power Co.
11.	Corporate Address	231 West Michigan Milwaukee, Wisconsin
12.	Corporate Contact	S. Burstein
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	R. F. Warnick
19.	IE Resident Inspector	W. Guldemond
20.	Licensing Project Manager	C. M. Trammell
21.	Docket Number	50-266
22.	License Number/Date of Issue	DPR-24 issued 10/05/70
23.	Public Document Room	Joseph Mann Public Library Two Rivers, Wisconsin Manitowoc Public Library Manitowoc, Wisconsin (Temporary LPDR for spent fuel pool capacity increase amendment only

P-10

INSPECTION STATUS

ENFORCEMENT STATUS

THE LICENSEE FAILED TO CONTROL ACCESS TO A VITAL AREA.
(8010 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

The unit returned to operation on August 9, 1980, following an outage for steam generator hydrostatic testing and eddy current examination of S/G tubes as required by the NRC order.

LAST IE SITE INSPECTION DATE: July 17-22, 1980

INSPECTION REPORT NO: 80-13

SUMMARY

INSPECTION SUMMARY

Inspection on June 4-27 and July 8, (80-10): Routine, unannounced inspection conducted by resident inspectors during the period June 4-27 and July 8, 1980, of operational safety verification, licensee event report followup, IE Bulletin followup, and IE Circular followup. The inspection involved 180 inspector-hours onsite by two NRC inspectors. An Enforcement Conference was held on July 8, 1980, to discuss the recent increase in personnel errors. The meeting involved 12 manhours by 4 NRC representatives. Of the four areas inspected, no items of noncompliance were identified in two areas. One item of noncompliance was identified in one area (uncontrolled vital area access).

Inspection on June 30 through July 30, (80-12): Licensee event followup; IE Bulletin followup; IE Circular followup; operational safety verification; administrative controls on containment purging; training staff qualification; review of periodic and special reports; monthly maintenance observation; monthly surveillance observation. The inspection involved 195 inspector-hours onsite by four NRC inspectors including 8 inspector-hours onsite during off-shifts. Of the nine areas inspected, one item of noncompliance was identified in one area (improper controlled area access and improper control of radioactive material).

Inspection on July 17 and 22, (80-13): Confirmatory Measurements including telephone discussion and closeout of results of a previous sample collection and the submission of spiked samples. No items of noncompliance or deviations were identified.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

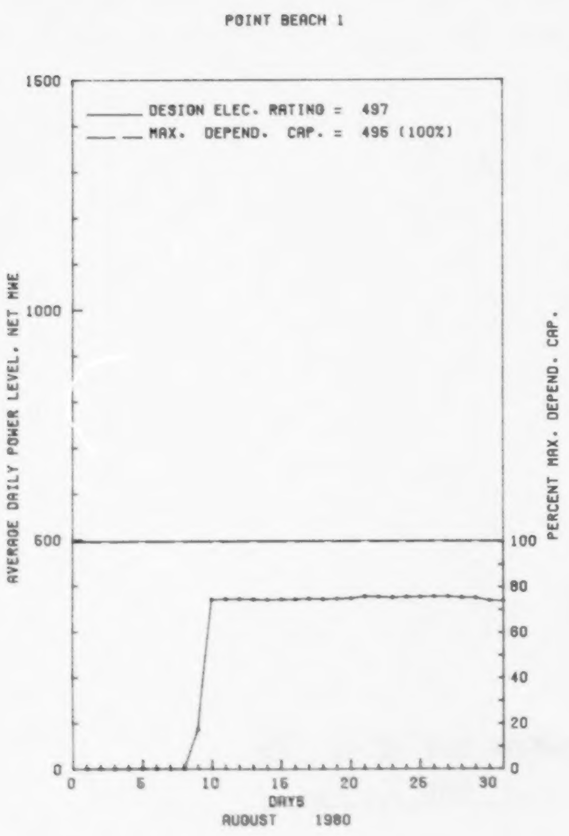
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-09/ 01T-0	07/28/80	08/11/80	Two dripping explosive plugs and two wet end plugs in the "A" S/G and one dripping tube and one wet end plug in the "B" S/G.
			Annual results and data report for 1978. (Special Report).

REVIEWED BY OMPA LOEB: T.C. Cintula **DATE:** 10·6·80

REVIEWED BY NRR DL: PCWagner **DATE:** 10/8/80

POINT BEACH 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-266
2. Utility Contact: C.W. FAY (414) 277-2811
3. Licensed Thermal Power (MWt): 1518
4. Nameplate Rating (Gross MWe): 582 x 0.9 = 524
5. Design Electrical Rating (Net MWe): 497
6. Maximum Dependable Capacity (Gross MWe): 519
7. Maximum Dependable Capacity (Net MWe): 495
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): 390
10. Reasons for Restrictions, If Any:
INHIBIT CORROSION OF STEAM GEN. TUBES.

ITEM 21 CALCULATED BY USING A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	5,855.8	86,087.0
12. Hours Reactor Was Critical	542.3	4,760.8	71,277.3
13. Reactor Reserve Shutdown Hours	.4	19.5	601.7
14. Hours Generator On-Line	536.9	4,610.2	69,004.2
15. Unit Reserve Shutdown Hours	1.3	152.8	755.2
16. Gross Thermal Energy (MWH)	650,041	5,558,066	96,310,861
17. Gross Electrical Energy (MWH)	209,220	1,807,220	32,440,970
18. Net Electrical Energy (MWH)	197,101	1,706,515	30,896,693
19. Unit Service Factor	72.2	78.7	80.2
20. Unit Availability Factor	72.3	81.3	81.0
*21. Unit Capacity Factor (MDC Net)	53.5	58.9	73.7
22. Unit Capacity Factor (DER Net)	53.3	58.6	72.2
23. Unit Forced Outage Rate	.0	.1	3.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
REFUELING - 11/07/80 - 1 MONTH.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
4	07/26/80	S	207.1	D	4	80-009	CB	HTEXCH	AS REQUIRED BY NRC CONFIRMATORY ORDER OF 11/30/79 (AMENDED 04/04/80), THE UNIT WAS TAKEN OFF LINE FOR THE 90 EFFECTIVE FULL POWER DAY STEAM GENERATOR TESTING. A 100% INSPECTION THROUGH THE FIRST SUPPORT WAS CONDUCTED IN BOTH STEAM GENERATORS. A TOTAL OF 28 CREVICE INDICATIONS AND TWO ABOVE THE TUBESHEET WERE FOUND IN THE "A" STEAM GENERATOR AND 22 CREVICE INDICATIONS WERE FOUND IN "B".

SUMMARY

POINT BEACH 1 CONCLUDED AN NRC REQUIRED OUTAGE FOR STEAM GENERATOR TESTING ON 8/9 AND OPERATED AT NEAR FULL POWER FOR THE REMAINDER OF THE MONTH.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

POINT BEACH 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Wisconsin/Manitowoc 15 miles N of Manitowoc
2.	Electric Reliability Council	Mid-America Interpool Network
3.	Type of Reactor	PWR
4.	Power Level (MWT)	1518
5.	Design Electrical Rating (MWE Net)	497
6.	Date of Initial Criticality	May 30, 1972
7.	Date of Commercial Operation	October 1, 1972
8.	Condenser Cooling Method	Straight Through
9.	Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10.	Licensee	Wisconsin Electric Power Co.
11.	Corporate Address	231 West Michigan Milwaukee, Wisconsin
12.	Corporate Contact	S. Burstein
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	R. F. Warnick
19.	IE Resident Inspector	W. Guldemond
20.	Licensing Project Manager	C. M. Trammell
21.	Docket Number	50-301
22.	License Number/Date of Issue	DPR-27 issued 11/16/71
23.	Public Document Room	Joseph Mann Public Library Two Rivers, Wisconsin Manitowoc Public Library Manitowoc, Wisconsin (Temporary LPDR for spent fuel pool capacity increase amendment only

INSPECTION STATUS

ENFORCEMENT STATUS

TECHNICAL SPECIFICATION 15.3.3.A.1.D REQUIRES TWO RHR PUMPS TO BE OPERABLE FOR NORMAL REACTOR OPERATION. TS 15.3.3.A.2.B PERMITS ONE RHR PUMP TO BE OUT OF SERVICE DURING REACTOR OPERATION UNDER CERTAIN SPECIFIED CONDITIONS. CONTRARY TO THE ABOVE, ON JUNE 19, 1980, WHILE THE 3D EMERGENCY DIESEL GENERATOR WAS OUT OF SERVICE FOR ANNUAL MAINTENANCE (RENDERING THE A SAFEGUARDS TRAIN, INCLUDING RHR PUMP 2P10A, INOPERABLE DURING A LOSS OF AC POWER EVENT) THE SUCTION VALVE FOR RHR PUMP 2P10B WAS CYCLED FROM FULL OPEN TO FULL CLOSED AND BACK TO FULL OPEN. FOR APPROXIMATELY 90 SECONDS, WHILE THE SUCTION VALVE WAS BEING CYCLED, BOTH RHR PUMPS WOULD HAVE BEEN INOPERABLE HAD A LOSS OF AC POWER EVENT OCCURRED.
(8010 INFRACTION)

TECHNICAL SPECIFICATION 15.3.7, ITEM B.1, PARAGRAPH D, STATES: "EITHER BUS A03 OR A04 MAY BE OUT OF SERVICE FOR A PERIOD NOT EXCEEDING 7 DAYS PROVIDED BOTH DIESEL GENERATORS ARE OPERABLE AND THE ASSOCIATED DIESEL GENERATOR IS OPERATING AND PROVIDING POWER TO THE ENGINEERED SAFEGUARD BUS NORMALLY SUPPLIED BY THE OUT OF SERVICE BUS." THE BASIS TO THE TECHNICAL SPECIFICATION STATES IN PART: "SEPARATION IS MAINTAINED IN THE 4160-VOLT SYSTEM TO ALLOW THE PLANT AUXILIARY EQUIPMENT TO BE ARRANGED ELECTRICALLY SO THAT REDUNDANT ITEMS RECEIVE THEIR POWER FROM THE TWO DIFFERENT BUSES. FOR EXAMPLE, THE SAFETY INJECTION PUMPS ARE SUPPLIED FROM THE 4160 VOLT BUSES 1-A05 AND 1-A06 FOR UNIT NO. 1 AND 2-A05 AND 2-A06 FOR UNIT NO. 2..." GENERAL DESIGN CRITERION 17 STATES IN PART: "THE ONSITE ELECTRIC POWER SUPPLIES...SHALL HAVE SUFFICIENT INDEPENDENCE, REDUNDANCY, AND TESTABILITY TO PERFORM THEIR SAFETY FUNCTIONS ASSUMING A SINGLE FAILURE." (ALSO ASSUMING A LOSS OF OFFSITE POWER). CONTRARY TO THE ABOVE, ENGINEERED SAFEGUARDS BUSES 2-A05 AND 2-A06 WERE ELECTRICALLY CONNECTED AND BOTH WERE SUPPLIED BY 4160 VOLT BUS A-04. THIS CONDITION WAS IDENTIFIED ON JUNE 9, 1980, BY THE LICENSEE AND IS BELIEVED TO HAVE EXISTED SINCE MAY 2, 1980 FOLLOWING A LOSS OF AC POWER TEST.
(8010 INFRACTION)

THE LICENSEE FAILED TO CONTROL ACCESS TO A VITAL AREA.
(8010 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

The unit operated routinely during the month.

LAST IE SITE INSPECTION DATE: July 17-22, 1980

INSPECTION REPORT NO: 80-13

SUMMARY

INSPECTION SUMMARY

Inspection on June 4-27 and July 8, (80-10): Routine, unannounced inspection conducted by resident inspectors during the period June 4-27 and July 8, 1980, of operational safety verification, licensee event report followup, IE Bulletin followup, and IE Circular followup. The inspection involved 180 inspector-hours onsite by two NRC inspectors. An Enforcement Conference was held on July 8, 1980, to discuss the recent increase in personnel errors. The meeting involved 12 manhours by 4 NRC representatives. Of the four areas inspected, no items of noncompliance were identified in two areas. Two items of noncompliance were identified in one area (RHR pumps operability impaired; ESF buses cross-connected). One item of noncompliance was identified in one area (uncontrolled vital area access).

Inspection on June 30 through July 30, (80-12): Licensee event followup; IE Bulletin followup; IE Circular followup; operational safety verification; administrative controls on containment purging; training staff qualification; review of periodic and special reports; monthly maintenance observation; monthly surveillance observation. The inspection involved 195 inspector-hours onsite by four NRC inspectors including 8 inspector-hours onsite during off-shifts. Of the nine areas inspected, one item of noncompliance was identified in one area (improper controlled area access and improper control of radioactive material).

Inspection on July 17 and 22, (80-13): Confirmatory Measurements including telephone discussion and closeout of results of a previous sample collection and the submission of spiked samples. No items of noncompliance or deviations were identified.

REPORTS RECEIVED FROM LICENSEE

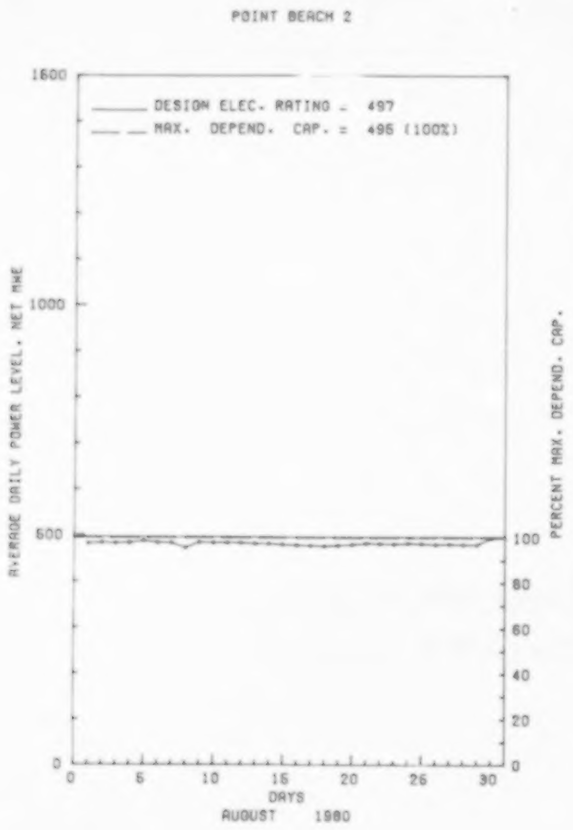
NUMBER	DATE OF EVENT	DATE OF REPORT	TYPE OF REPORT	SUBJECT
	NONE			

REVIEWED BY OMPA LOEB: T. C. Cintula **DATE:** 10·6·80

REVIEWED BY NRR DL: P C Wagner **DATE:** 10/8/80

POINT BEACH 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-301
2. Utility Contact: C.W. FAY (414) 277-2811
3. Licensed Thermal Power (MWt): 1518
4. Nameplate Rating (Gross MWe): 582 x 0.9 = 524
5. Design Electrical Rating (Net MWe): 497
6. Maximum Dependable Capacity (Gross MWe): 519
7. Maximum Dependable Capacity (Net MWe): 495
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

ITEM 21 CALCULATED BY USING A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	5,855.0	76,872.0
12. Hours Reactor Was Critical	744.0	4,754.9	63,651.8
13. Reactor Reserve Shutdown Hours	.0	19.2	185.3
14. Hours Generator On-Line	744.0	4,688.7	67,415.7
15. Unit Reserve Shutdown Hours	.0	13.5	119.9
16. Gross Thermal Energy (MWH)	1,124,200	6,873,370	84,525,906
17. Gross Electrical Energy (MWH)	376,080	2,298,650	28,659,990
18. Net Electrical Energy (MWH)	358,959	2,181,744	27,263,873
19. Unit Service Factor	100.0	80.1	88.1
20. Unit Availability Factor	100.0	80.3	88.2
*21. Unit Capacity Factor (MDC Net)	97.5	75.5	78.4
22. Unit Capacity Factor (DER Net)	97.1	75.2	77.4
23. Unit Forced Outage Rate	.0	7.0	1.9

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	NONE								

SUMMARY

POINT BEACH 2 OPERATED AT NEAR FULL POWER FOR THE ENTIRE MONTH, WITH NO POWER OUTAGES OR REDUCTIONS.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continued
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

PRAIRIE ISLAND 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Minnesota/Goodhue 28 miles SE of Minneapolis
2.	Electric Reliability Council	Mid Continent Area Reliability Coordination Agreement
3.	Type of Reactor	PWR
4.	Power Level (MWT)	1650
5.	Design Electrical Rating (MWE Net)	530
6.	Date of Initial Criticality	December 1, 1973
7.	Date of Commercial Operation	December 16, 1973
8.	Condenser Cooling Method	Cooling Towers
9.	Condenser Cooling Water Source	Mississippi River

Utility & Contractor Information

10.	Licensee	Northern States Power Company
11.	Corporate Address	414 Nicollet Mall - 8th Floor Minneapolis, Minnesota 55401
12.	Corporate Contact	G. H. Neils (Oper.) L. O. Mayer (Lic.)
13.	Architect/Engineer	Pioneer Services & Eng. Co.
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Northern States Power Co.

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	W. S. Little
19.	IE Resident Inspector	C. D. Feierabend 612-388-8209
20.	Licensing Project Manager	R. Martin
21.	Docket Number	50-282
22.	License Number/Date of Issue	DPR-42 issued 8/9/73
23.	Public Document Room	Environmental Conservation Library Minneapolis Public Library 300 Nicollet Mall Minneapolis, Minnesota 55401

INSPECTION STATUS

ENFORCEMENT STATUS

TECHNICAL SPECIFICATION 6.5.B.1 REQUIRES THAT INDIVIDUALS PERMITTED TO ENTER A HIGH RADIATION AREA BE PROVIDED WITH A RADIATION MONITORING DEVICE WHICH CONTINUOUSLY INDICATES THE RADIATION DOSE RATE IN THE AREA. CONTRARY TO THE ABOVE, SEVERAL OPERATIONS PERSONNEL STATED THAT THEY DID NOT ROUTINELY TAKE A DOSE RATE INDICATING DEVICE INTO HIGH RADIATION AREAS WHEN DRESSING IN FULL ANTICONTAMINATION CLOTHING. (8008 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

FACILITY ITEMS

A significant weakness was identified with radiation survey instrument usage. (80-08)

MANAGERIAL ITEMS

A significant weakness was identified with offshift radiation protection coverage. (80-08)

PLANT STATUS

Refueling outage began on August 31, 1980.

LAST IE SITE INSPECTION DATE: June 1 through July 31, 1980

INSPECTION REPORT NO: 80-13

SUMMARY

INSPECTION SUMMARY

Appraisal on May 5-15, (80-08): Special, announced appraisal of health physics program, including organization and management, qualifications, training, internal and external exposure controls, surveys and access controls, instrumentation, ALARA, radioactive waste, facilities and equipment, and accident response. The appraisal involved 345 man-hours onsite by four inspectors. Two significant weaknesses in the health physics program were identified. These weaknesses are in the areas of offshift staffing and survey instrument use practices. One apparent item of noncompliance was found (inadequate radiation survey instrument use in high radiation areas).

Inspection on June 1 through July 30, (80-13): Routine, resident inspection of plant operation, maintenance, surveillance, security, radiation protection, followup of licensee reported events, followup on IE Bulletins, preparation for refueling, preparation of irradiated specimen for shipment, quality control and training. The inspection involved 175 inspector-hours onsite by two NRC inspectors. No items of noncompliance were identified. One item concerning quality control remains unresolved.

REPORTS RECEIVED FROM LICENSEE

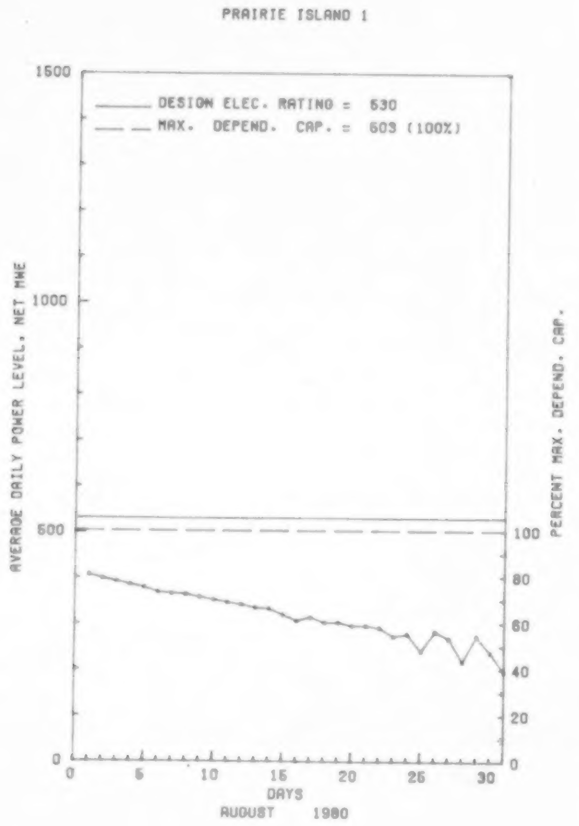
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-19/ 03L-0	06/30/80	07/24/80	Two of four setpoints were set as much as 0.088% above TS limit of 108%.
80-20	07/15/80	07/16/80	Severe electrical storm caused operation of several breakers in substation.
80-24/ 03L-0	07/07/80	07/07/80	Failure of reactor recirculation loop sample outboard isolation valve to close.

REVIEWED BY OMPA LOEB: T.C. Cintula **DATE:** 10·6·80

REVIEWED BY NRR DL: R E Martin **DATE:** 10/8/80

PRAIRIE ISLAND 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-282
2. Utility Contact: A. A. Hunstad (612) 388-4767
3. Licensed Thermal Power (MWt): 1650
4. Nameplate Rating (Gross MWe): 659 x 0.9 = 593
5. Design Electrical Rating (Net MWe): 530
6. Maximum Dependable Capacity (Gross MWe): 534
7. Maximum Dependable Capacity (Net MWe): 503
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, if Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	5,855.0	58,823.0
12. Hours Reactor Was Critical	740.9	5,256.4	46,454.8
13. Reactor Reserve Shutdown Hours	.0	.0	5,531.9
14. Hours Generator On-Line	740.8	5,246.5	45,370.1
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	904,020	8,140,749	70,219,743
17. Gross Electrical Energy (MWH)	259,500	2,493,680	22,647,350
18. Net Electrical Energy (MWH)	236,799	2,327,064	21,166,316
19. Unit Service Factor	99.6	89.6	77.1
20. Unit Availability Factor	99.6	89.6	77.1
21. Unit Capacity Factor (MDC Net)	63.3	79.0	71.5
22. Unit Capacity Factor (DER Net)	60.1	75.0	67.9
23. Unit Forced Outage Rate	.0	3.2	12.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 11/02/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	08/31/80	S	3.2	C	2		RC	FUELXX	REFUELING OUTAGE COMMENCES.

SUMMARY

PRAIRIE ISLAND 1 OPERATED IN A COAST DOWN CONDITION FOR THE MONTH, WITH A REFUELING OUTAGE COMMENCING ON AUGUST 31ST.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

PRAIRIE ISLAND 2

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Minnesota/Goodhue 28 miles SE of Minneapolis
2.	Electric Reliability Council	Mid-Continent Area Reliability Coordination Agreement
3.	Type of Reactor	PWR
4.	Power Level (MWT)	1650
5.	Design Electrical Rating (MWE Net)	530
6.	Date of Initial Criticality	December 17, 1974
7.	Date of Commercial Operation	December 21, 1974
8.	Condenser Cooling Method	Cooling Towers
9.	Condenser Cooling Water Source	Mississippi River

Utility & Contractor Information

10.	Licensee	Northern States Power Company
11.	Corporate Address	414 Nicollet Mall - 8th Floor Minneapolis, Minnesota 55401
12.	Corporate Contact	G. H. Neils (Oper.) L. O. Mayer (Lic.)
13.	Architect/Engineer	Pioneer Services & Eng. Co.
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Northern States Power Co.

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	W. S. Little
19.	IE Resident Inspector	C. D. Feierabend 612-388-8209
20.	Licensing Project Manager	R. Martin
21.	Docket Number	50-306
22.	License Number/Date of Issue	DPR-60 issued 10/29/74
23.	Public Document Room	Environmental Conservation Library Minneapolis Public Library 300 Nicollet Mall Minneapolis, Minnesota 55401

INSPECTION STATUS

ENFORCEMENT STATUS

TECHNICAL SPECIFICATION 6.5.B.1 REQUIRES THAT INDIVIDUALS PERMITTED TO ENTER A HIGH RADIATION AREA BE PROVIDED WITH A RADIATION MONITORING DEVICE WHICH CONTINUOUSLY INDICATES THE RADIATION DOSE RATE IN THE AREA. CONTRARY TO THE ABOVE, SEVERAL OPERATIONS PERSONNEL STATED THAT THEY DID NOT ROUTINELY TAKE A DOSE RATE INDICATING DEVICE INTO HIGH RADIATION AREAS WHEN DRESSING IN FULL ANTICONTAMINATION CLOTHING. (8009 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

FACILITY ITEMS

A significant weakness was identified with radiation survey instrument usage. (80-09)

MANAGERIAL ITEMS

A significant weakness was identified with offshift radiation protection coverage. (80-09)

PLANT STATUS

Operating routinely.

LAST IE SITE INSPECTION DATE: June 1 through July 31, 1980

INSPECTION REPORT NO: 80-13

SUMMARY

INSPECTION SUMMARY

Appraisal on May 5-15, (80-09): Special, announced appraisal of health physics program, including organization and management, qualifications, training, internal and external exposure controls, surveys and access controls, instrumentation, ALARA, radioactive waste, facilities and equipment, and accident response. The appraisal involved 345 man-hours onsite by four inspectors. Two significant weaknesses in the health physics program were identified. These weaknesses are in the areas of offshift staffing and survey instrument use practices. One apparent item of noncompliance was found (inadequate radiation survey instrument use in high radiation areas).

Inspection on June 1 through July 30, (80-13): Routine, resident inspection of plant operation, maintenance, surveillance, security, radiation protection, followup of licensee reported events, followup on IE Bulletins, preparation for refueling, preparation of irradiated specimen for shipment, quality control and training. The inspection involved 175 inspector-hours onsite by two NRC inspectors. No items of noncompliance were identified. One item concerning quality control remains unresolved.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

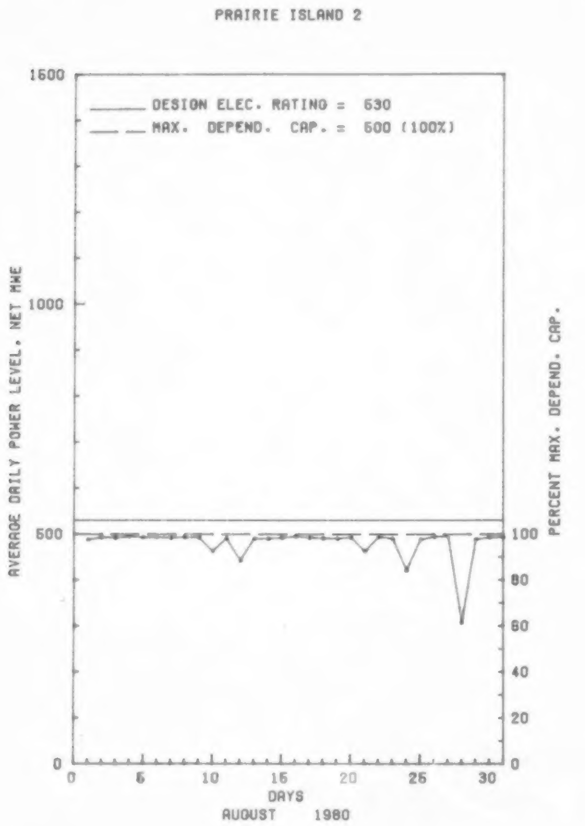
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
None.			

REVIEWED BY OMPA LOEB: T. C. Cintula **DATE:** 10·6·80

REVIEWED BY NRR DL: R E Martin **DATE:** 10/8/80

PRAIRIE ISLAND 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-306
2. Utility Contact: A. A. Hunstad (612) 388-4767
3. Licensed Thermal Power (MWt): 1650
4. Nameplate Rating (Gross MWe): 659 x 0.9 = 593
5. Design Electrical Rating (Net MWe): 530
6. Maximum Dependable Capacity (Gross MWe): 531
7. Maximum Dependable Capacity (Net MWe): 500
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	5,855.0	49,941.0
12. Hours Reactor Was Critical	744.0	4,546.1	43,675.1
13. Reactor Reserve Shutdown Hours	.0	89.4	1,495.4
14. Hours Generator On-Line	744.0	4,474.5	42,875.5
15. Unit Reserve Shutdown Hours	.0	.0	0
16. Gross Thermal Energy (MWH)	1,193,382	7,072,132	66,744,635
17. Gross Electrical Energy (MWH)	380,540	2,274,470	21,328,600
18. Net Electrical Energy (MWH)	357,283	2,125,851	19,964,483
19. Unit Service Factor	100.0	76.4	85.9
20. Unit Availability Factor	100.0	76.4	85.9
21. Unit Capacity Factor (MDC Net)	96.0	72.6	80.0
22. Unit Capacity Factor (DER Net)	90.6	68.5	75.4
23. Unit Forced Outage Rate	.0	.5	4.4

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUELING, JANUARY 1981, 6 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	08/28/80	S	.0	F	5				LOAD DEPRESSED DUE TO TEMPERATURE LIMITATIONS ENCOUNTERED DURING CHLORINATION OF THE CIRCULATING WATER SYSTEM.

SUMMARY

PRAIRIE ISLAND 2 REMAINED ON LINE FOR THE ENTIRE MONTH, WITH ONE REDUCTION FOR TEMPERATURE LIMITATIONS DURING CHLORINATION OF THE CIRCULATING WATER SYSTEM.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

QUAD CITIES 1

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Illinois/Rock Island 20 miles NE of Moline
2.	Electric Reliability Council	Mid-America Interpool Network
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2511
5.	Design Electrical Rating (MWE Net)	789
6.	Date of Initial Criticality	October 18, 1971
7.	Date of Commercial Operation	February 18, 1973
8.	Condenser Cooling Method	Spray Canal
9.	Condenser Cooling Water Source	Mississippi River

Utility & Contractor Information

10.	Licensee	Commonwealth Edison Co.
11.	Corporate Address	P. O. Box 767 Chicago, Illinois 60690
12.	Corporate Contact	Cordell Reed,
13.	Architect/Engineer	Sargent and Lundy
14.	Nuclear Steam Supply System	GE
15.	Constructor	United Engineers & Constructors, Inc.

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	R. Spessard
19.	IE Resident Inspector	N. J. Chrissotimos (309) 654-2227
20.	Licensing Project Manager	R. B. Bevan
21.	Docket Number	50-254
22.	License Number/Date of Issue	DPR-29 issued 10/1/71
23.	Public Document Room	Moline Public Library 504 17th Street Moline, Illinois 61265

INSPECTION STATUS

ENFORCEMENT STATUS

AS REQUIRED BY 10 CFR 71.5, NO LICENSEE SHALL TRANSPORT ANY LICENSED MATERIAL OUTSIDE OF THE CONFINES OF HIS PLANT OR OTHER PLACE OF USE, UNLESS THE LICENSEE COMPLIES WITH THE APPLICABLE REQUIREMENTS OF THE REGULATIONS APPROPRIATE TO THE MODE OF TRANSPORT OF THE DEPARTMENT OF TRANSPORTATION IN 49 CFR PARTS 170-189. CONTRARY TO THE ABOVE, ON DECEMBER 19, 1979, THE DAY OF THE INSPECTION, A CASK SHIPPED FROM YOUR QUAD-CITIES FACILITY WAS NOT MARKED "RADIOACTIVE LSA" AS REQUIRED BY 49 CFR 173.392(C)(8).
(7932 DEFICIENCY)

10 CFR 70.51(B)(1) STATES THAT EACH LICENSEE SHALL KEEP RECORDS SHOWING THE RECEIPT, INVENTORY, (INCLUDING LOCATIONS), DISPOSAL, ACQUISITION, AND TRANSFER OF ALL SPECIAL NUCLEAR MATERIAL IN HIS POSSESSION, REGARDLESS OF ITS ORIGIN OR METHOD OF ACQUISITION. CONTRARY TO THE ABOVE, THE UPDATE OF THE FUEL LOCATION AND EXPOSURE RECORDS (FORM E) WAS NOT CORRECTLY ACCOMPLISHED AND REFLECTED MANY FUEL ASSEMBLIES IN THE UNIT 1 FUEL POOL LOCATIONS, WHEN IN FACT, ON JUNE 3, 1980, THESE FUEL ASSEMBLIES HAD BEEN TRANSFERRED FROM THE UNIT 1 FUEL POOL TO THE UNIT 2 FUEL POOL LOCATIONS.
(8016 DEFICIENCY)

10 CFR PART 70.51(C) REQUIRES THE LICENSEE TO ESTABLISH, MAINTAIN AND FOLLOW WRITTEN MATERIAL CONTROL AND ACCOUNTING PROCEDURES WHICH ARE SUFFICIENT TO ENABLE THE LICENSEE TO ACCOUNT FOR THE SPECIAL NUCLEAR MATERIAL IN HIS POSSESSION UNDER LICENSE. CONTRARY TO THE ABOVE, THE LICENSEE'S PROCEDURES WERE NOT SUFFICIENT TO PHYSICALLY ACCOUNT FOR AND CONTROL FISSION COUNTERS AS A PART OF PHYSICAL INVENTORY. AS A RESULT OF AN INVENTORY VERIFICATION CONDUCTED BY THE INSPECTION TEAM, ONE OF EIGHT FISSION COUNTERS (DUNKERS) WAS DISCOVERED TO BE MISSING. THIS GENERAL ELECTRIC FISSION COUNTER SERIAL NUMBER 6554075, CONTAINED APPROXIMATELY 1.9 GRAMS OF URANIUM 235 ENRICHED AT 93%.
(8016 INFRACTION)

THE LICENSEE FAILED TO CHECK TWO VITAL AREA DOORS.
(8018 DEFICIENCY)

THE LICENSEE FAILED TO CONTROL ACCESS TO A VITAL AREA.
(8018 INFRACTION)

THE LICENSEE FAILED TO PROVIDE ADEQUATE KEY CONTROL.
(8018 DEFICIENCY)

THE LICENSEE FAILED TO ESTABLISH AND PROPERLY IMPLEMENT PROCEDURES TO COMPENSATE FOR THE FAILURE OF SPECIFIC SECURITY RELATED EQUIPMENT.
(8018 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

FACILITY ITEMS

Testing of Quad-Cities 1 as required by IE Bulletin 80-17 was completed with no major discrepancies between as-built and as-designed scram discharge systems. All systems functioned as designed. (80-12)

PLANT STATUS

The unit was shut down on August 31, 1980 to begin a scheduled 12 week refueling outage.

LAST IE SITE INSPECTION DATE: July 14-18, 1980

INSPECTION REPORT NO: 80-18

SUMMARY

INSPECTION SUMMARY

Inspection on June 6 through July 18, (80-12): Routine, unannounced inspection consisting of operational safety verification, maintenance observation, licensee event report followup, IE Bulletin followup, plant trips, procedure verification, followup on Headquarters requests and independent inspection. The inspection involved 180 inspector-hours onsite by two NRC inspectors including 40 inspector-hours onsite during offshifts. No items of noncompliance were identified.

Inspection on May 28, (80-17): During the inspection of radioactive waste at Richland, Washington from May 27 to May 30, 1980, trailer 544142 was inspected. A representative of the State of Washington also examined the shipment. One apparent item of noncompliance with DOT regulations was identified (failure to meet strong, tight package requirement). It appears that the Department of Transportation and the NRC Region III Office are the appropriate organizations to take enforcement actions.

Inspection on July 14-18, (80-18): Included a review of the Security Plan; Security Program Audit; Security Organization; Protected Area Physical Barriers; Vital Area Physical Barriers; Lighting; Access Control (Identification, Authorization, and Badging); Access Control (Search); Access Control (Escorting); Access Control (Vital Areas); Alarm Stations; Detection Aids; Assessment Aids; Communications; Testing and Maintenance; Compensatory Measures; Power Supply; Response; Locks, Keys, and Combinations; Records and Reports; Licensee Action on Previous Inspection Findings; and Independent Inspection Effort. The inspection involved 127.5 inspector-hours onsite by three NRC inspectors. The inspection began during the day shift; 24 inspector-hours were accomplished during offshift periods. The licensee was found to be in compliance with NRC requirements within the areas examined during the inspection, except for the following four items: Compensatory Measures - The licensee failed to establish and properly implement procedures to compensate for the failure of specific security related equipment; Access Control (Vital Areas) - The licensee failed to control access to a vital area; Access Control (Physical Barriers) - The licensee failed to check two vital area doors; and Locks, Keys, and Combinations - The licensee failed to adequately control blank key cards.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

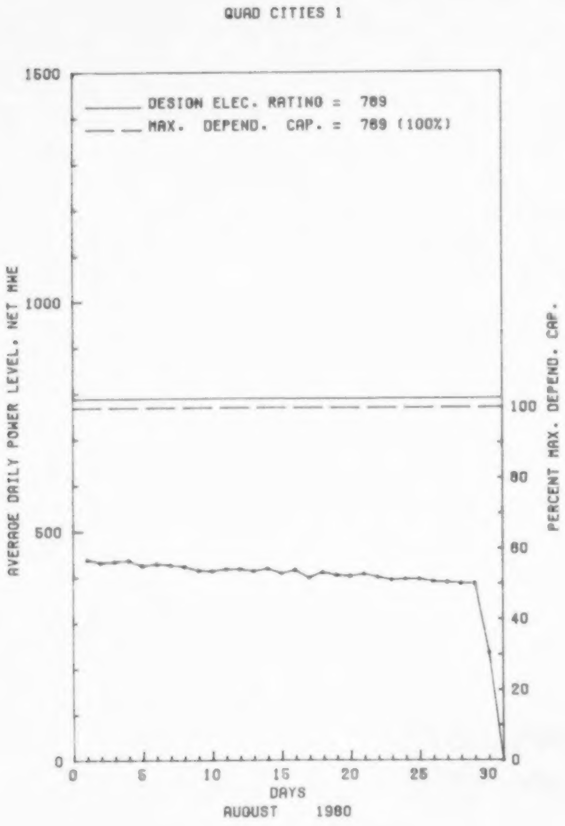
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-17/ 03L-0	07/16/80	08/06/80	Vacuum breaker valve AO-1-1601-20B was open when it should have been closed.
80-18/ 03L-0	08/05/80	08/15/80	Unit 1 was at 486 MWe and Unit 2 at 698 MWe. the main chimney off gas sample pump tripped and the backup pump also tripped.

REVIEWED BY OMPA LOEB: John h Crooks **DATE:** 10/6/80

REVIEWED BY NRR DL: RB Bevan **DATE:** 10/8/80

QUAD CITIES 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-254
2. Utility Contact: R. TUBBS (309) 654-2241 X176
3. Licensed Thermal Power (MWt): 2511
4. Nameplate Rating (Gross MWe): 920 x 0.9 = 828
5. Design Electrical Rating (Net MWe): 789
6. Maximum Dependable Capacity (Gross MWe): 813
7. Maximum Dependable Capacity (Net MWe): 769
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	5,855.0	72,815.0
12. Hours Reactor Was Critical	723.4	5,725.1	60,439.2
13. Reactor Reserve Shutdown Hours	.0	.0	3,421.9
14. Hours Generator On-Line	720.1	5,623.1	57,664.0
15. Unit Reserve Shutdown Hours	.0	.0	909.2
16. Gross Thermal Energy (MWH)	1,077,710	11,394,718	115,938,699
17. Gross Electrical Energy (MWH)	329,718	3,662,730	37,285,784
18. Net Electrical Energy (MWH)	291,109	3,385,609	34,801,145
19. Unit Service Factor	96.8	96.0	79.2
20. Unit Availability Factor	96.8	96.0	80.4
21. Unit Capacity Factor (MDC Net)	50.9	75.2	62.2
22. Unit Capacity Factor (DER Net)	49.6	73.3	60.6
23. Unit Forced Outage Rate	.0	3.6	7.4

24. Shutdowns Scheduled Over Next 6 Months (Type, Data, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 11/22/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
12	08/30/80	S	.0	F	5				LOAD REDUCTION DUE TO SECURING OF REACTOR FEED PUMP.
13	08/31/80	S	23.9	C	2		RC	FUELXX	GENERATOR REMOVED FROM ON-LINE STATUS FOR CYCLE V REFUELING OUTAGE. (REACTOR WAS MANUALLY SCRAMMED DUE TO FAILURE OF AN ELECTROMATIC RELIEF VALVE TO RESEAT).

SUMMARY

QUAD CITIES ONE OPERATED IN A COASTDOWN CONDITION FOR THE MONTH, AND WENT INTO SCHEDULED REFUELING FOLLOWING FAILURE OF AN ELECTROMATIC RELIEF VALVE TO RESEAT.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

QUAD CITIES 2

FACILITY DATA

Facility Description

1. Location (state/county) (nearest population center)	Illinois/Rock Island 20 miles NE of Moline
2. Electric Reliability Council	Mid-America Interpool Network
3. Type of Reactor	BWR
4. Power Level (MWT)	2511
5. Design Electrical Rating (MWE Net)	789
6. Date of Initial Criticality	April 26, 1972
7. Date of Commercial Operation	March 10, 1973
8. Condenser Cooling Method	Spray Canal
9. Condenser Cooling Water Source	Mississippi River

Utility & Contractor Information

10. Licensee	Commonwealth Edison Co.
11. Corporate Address	P. O. Box 767 Chicago, Illinois 60690
12. Corporate Contact	Cordell Reed,
13. Architect/Engineer	Sargent & Lundy
14. Nuclear Steam Supply System	GE
15. Constructor	United Engineers & Constructors, Inc.

Regulatory Information

16. IE Region responsible	III
17. IE Branch Chief	R. Heishman
18. IE Section Chief	R. Spessard
19. IE Resident Inspector	N. J. Chrissotimos (309) 654-2227
20. Licensing Project Manager	R. B. Bevan
21. Docket Number	50-265
22. License Number/Date of Issue	DPR-30 issued 3/21/72
23. Public Document Room	Moline Public Library 504 17th Street Moline, Illinois 61265

INSPECTION STATUS

ENFORCEMENT STATUS

THE LICENSEE FAILED TO CHECK TWO VITAL AREA DOORS.
(8020 DEFICIENCY)

THE LICENSEE FAILED TO CONTROL ACCESS TO A VITAL AREA.
(8020 INFRACTION)

THE LICENSEE FAILED TO PROVIDE ADEQUATE KEY CONTROL.
(8020 DEFICIENCY)

THE LICENSEE FAILED TO ESTABLISH AND PROPERLY IMPLEMENT PROCEDURES TO COMPENSATE FOR THE FAILURE OF SPECIFIC SECURITY RELATED EQUIPMENT.
(8020 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

FACILITY ITEMS

Testing of Quad-Cities 2 as required by IE Bulletin 80-17 was completed with no major discrepancies between as-built and as-designed scram discharge systems. All systems functioned as designed. (80-15)

PLANT STATUS

Plant operating routinely.

LAST IE SITE INSPECTION DATE: July 14-18, 1980

INSPECTION REPORT NO: 80-20

SUMMARY

INSPECTION SUMMARY

Inspection on June 6 through July 18, (80-15): Routine, unannounced inspection consisting of operational safety verification, maintenance observation, licensee event report followup, IE Bulletin followup, plant trips, procedure verification, followup on Headquarters requests and independent inspection. The inspection involved 180 inspector-hours onsite by two NRC inspectors including 40 inspector-hours onsite during offshifts. No items of noncompliance were identified.

Inspection on May 23, (80-19): During the inspection of radioactive waste at Richland, Washington from May 27 to May 30, 1980, trailer 544142 was inspected. A representative of the State of Washington also examined the shipment. One apparent item of noncompliance with DOT regulations was identified (failure to meet strong, tight package requirement). It appears that the Department of Transportation and the NRC Region III Office are the appropriate organizations to take enforcement actions.

Inspection on July 14-18, (80-20): Included a review of the Security Plan; Security Program Audit; Security Organization; Protected Area Physical Barriers; Vital Area Physical Barriers; Lighting; Access Control (Identification, Authorization, and Badging); Access Control (Search); Access Control (Escorting); Access Control (Vital Areas); Alarm Stations; Detection Aids; Assessment Aids; Communications; Testing and Maintenance; Compensatory Measures; Power Supply; Response; Locks, Keys, and Combinations; Records and Reports; Licensee Action on Previous Inspection Findings; and Independent Inspection Effort. The inspection involved 127.5 inspector-hours onsite by three NRC inspectors. The inspection began during the day shift; 24 inspector-hours were accomplished during offshift periods. The licensee was found to be in compliance with NRC requirements within the areas examined during the inspection, except for the following four items: Compensatory Measures - The licensee failed to establish and properly implement procedures to compensate for the failure of specific security related equipment; Access Control (Vital Areas) - The licensee failed to control access to a vital area; Access Control (Physical Barriers) - The licensee failed to check two vital area doors; and Locks, Keys, and Combinations - The licensee failed to adequately control blank key cards.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

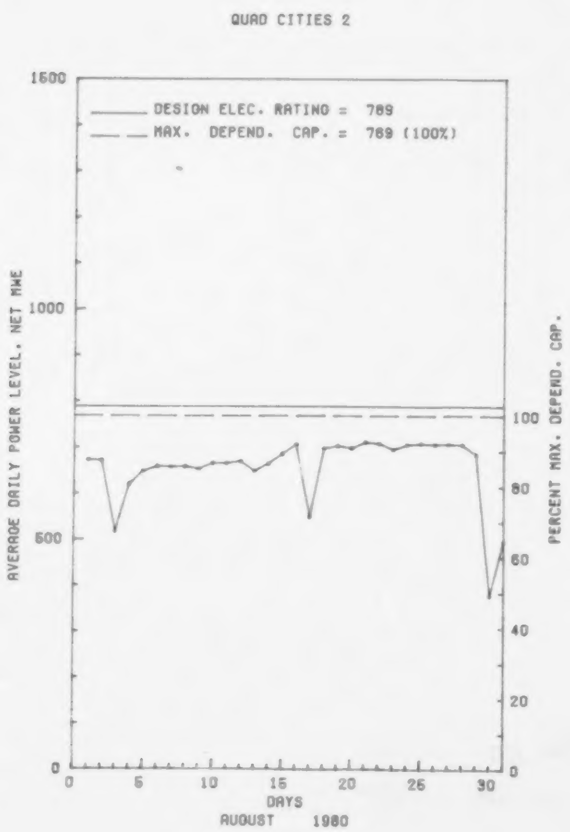
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-16/ 03L-0	07/23/80	08/11/80	During surveillance, pressure switch PS-2-261-30C tripped at 841 psig versus TS limit of not less than 850 psig.
80-17/ 03L-0	07/24/80	08/12/80	During surveillance, pressure switch PS-2-1622B actuated at 20.3 inches of water versus TS limit of less than 13.8 inches of water.

REVIEWED BY OMPA LOEB: John R Crooks **DATE:** 10/6/80

REVIEWED BY NRR DL: RBBevan **DATE:** 10/8/80

QUAD CITIES 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-265
2. Utility Contact: ROBERT TUBBS (309) 654-2241 X176
3. Licensed Thermal Power (MWt): 2511
4. Nameplate Rating (Gross MWe): 920 x 0.9 = 828
5. Design Electrical Rating (Net MWe): 789
6. Maximum Dependable Capacity (Gross MWe): 813
7. Maximum Dependable Capacity (Net MWe): 769
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	5,855.0	71,925.0
12. Hours Reactor Was Critical	744.0	3,061.5	56,190.4
13. Reactor Reserve Shutdown Hours	.0	.0	2,985.8
14. Hours Generator On-Line	744.0	2,942.0	53,733.8
15. Unit Reserve Shutdown Hours	.0	.0	702.9
16. Gross Thermal Energy (MWH)	1,660,264	6,455,056	109,471,163
17. Gross Electrical Energy (MWH)	512,083	2,017,686	34,880,736
18. Net Electrical Energy (MWH)	488,388	1,880,976	32,623,501
19. Unit Service Factor	100.0	50.2	74.7
20. Unit Availability Factor	100.0	50.2	75.7
21. Unit Capacity Factor (MDC Net)	85.4	41.8	59.0
22. Unit Capacity Factor (DER Net)	83.2	40.7	57.5
23. Unit Forced Outage Rate	.0	7.0	9.2

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
14	08/29/80	F	.0	H	5	80-19			LOAD REDUCTION DUE TO FAILURE OF 2A RECIRCULATION PUMP MG SET 4 KV METER TRANSFORMER.

SUMMARY

QUAD CITIES 2 REMAINED ON LINE FOR THE ENTIRE MONTH, WITH ONE REDUCTION ON 08/29/80 FOR RECIRCULATION PUMP METER TRANSFORMER PROBLEMS.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

RANCHO SECO

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	California/Sacramento 25 miles SE of Sacramento
2.	Electric Reliability Council	Western Systems Coordinating Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2772
5.	Design Electrical Rating (MWE Net)	918
6.	Date of Initial Criticality	September 16, 1974
7.	Date of Commercial Operation	April 17, 1975 (92.6% of full power)
8.	Condenser Cooling Method	Closed Cycle
9.	Condenser Cooling Water Source	Natural Draft Cooling Towers - Folsom Canal

Utility & Contractor Information

10.	Licensee	Sacramento Municipal Utility District
11.	Corporate Address	6201 South Street P. O. Box 15830 Sacramento, California
12.	Corporate Contact	J. J. Mattimoe Assistant General Manager, Chief Engineer
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	B&W
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	V
17.	IE Branch Chief	J. L. Crews
18.	IE Section Chief	B. H. Faulkenberry
19.	IE Resident Inspector	H. L. Canter (209) 748-2791
20.	Licensing Project Manager	D. J. Garner
21.	Docket Number	50-312
22.	License Number/Date of Issue	DPR-54, 8/16/74
23.	Public Document Room	Business and Municipal Department Sacramento City-County Library 828 I Street Sacramento, California 95814

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

SYSTEMS AND COMPONENT PROBLEMS:

PROBLEM WITH TURBINE AND COMPONENT COOLING WATER SYSTEM CAUSED APPROXIMATELY THREE WEEK OUTAGE DURING REPORT PERIOD.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

AWAITING DECISION OF ASLB. MEETING WITH SMUD MANAGEMENT AUGUST 26, 1980 FOR SALD PROGRAM.

PLANT STATUS:

UNIT SHUTDOWN TO REPLACE DAMAGED TURBINE THRUST BEARING.

LAST IE SITE INSPECTION DATE: 08/01-31/80+

INSPECTION REPORT NO. 50-312/80-26+

SUMMARY

INSPECTION SUMMARY

INSPECTION ON JULY 27-31, 1980 (REPORT NO. 50-312/80-22) AREAS INSPECTED: INCLUDED A REVIEW OF SECURITY PLAN; SECURITY ORGANIZATION; ACCESS CONTROL (IDENTIFICATION, AUTHORIZATION AND BADGING); ACCESS CONTROL (SEARCH); ACCESS CONTROL (VITAL AREAS); ALARM STATIONS; COMMUNICATIONS; POWER SUPPLY; RESPONSE; LOCKS, KEYS AND COMBINATIONS. THE INSPECTION INVOLVED 70 INSPECTOR HOURS ONSITE BY TWO NRC INSPECTORS

RESULTS: NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION ON AUGUST 18-22, 1980 (REPORT NO. 50-312/80-24) AREAS INSPECTED: ROUTINE, UNANNOUNCED INSPECTION OF AUDIT IMPLEMENTATION; SURVEILLANCE PROGRAM; FOLLOWUP ON PREVIOUSLY IDENTIFIED ITEMS; FOLLOWUP ON IE BULLETINS AND CIRCULARS; AND INDEPENDENT INSPECTION EFFORT. THE INSPECTION INVOLVED 35 INSPECTOR HOURS ONSITE BY ONE NRC INSPECTOR.

RESULTS: OF THE FIVE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN FOUR AREAS. ONE ITEM OF NONCOMPLIANCE (INFRACTION - FAILURE TO AUDIT CONFORMANCE WITH CERTAIN TECHNICAL SPECIFICATION REQUIREMENTS) AND ONE DEVIATION (FAILURE OF MSRC TO REVIEW AUDIT REPORTS AT A SCHEDULED MEETING IN A TIMELY MANNER) WERE IDENTIFIED IN ONE AREA.

INSPECTION ON AUGUST 11-15, 1980 (REPORT NO. 50-312/80-25) AREAS INSPECTED: ROUTINE, UNANNOUNCED INSPECTION BY REGIONAL BASED INSPECTOR OF THE LICENSEE'S ACTIVITIES PERFORMED IN RESPONSE TO OPEN ITEMS FROM PREVIOUS INSPECTIONS AND INSERVICE INSPECTION DATA REVIEW. THE INSPECTION INVOLVED 40 INSPECTOR HOURS ONSITE BY ONE NRC INSPECTOR.

RESULTS: NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION ON AUGUST 1-31, 1980 (REPORT NO. 50-312/80-26) REPORT BEING PREPARED; TO BE REPORTED NEXT MONTH

NRC POWER RESTRICTIONS: None

REPORTS RECEIVED FROM LICENSEE

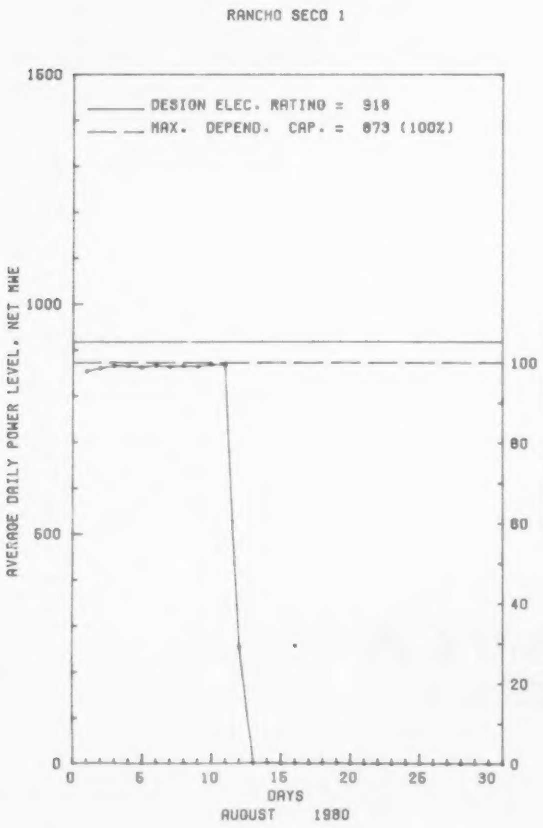
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-33/ 04T-0	06-30-80	07-25-80	APPENDIX B, SECTION 5.6.2.C NORTH AND WEST PERIMETER FENCE DOSE.
80-34/ 04L-0	06-02-80	07-30-80	EXCEEDED PH LIMIT IN EFFLUENT FOR 15 MINUTES.
80-35/ 04L-0	07-03-80	08-06-80	EXCEEDED EFFLUENT PH LIMIT.
80-36/ 04L-0	07-10-80	08-14-80	EXCEEDED EFFLUENT PH LIMIT.
80-37/ 04L-0	07-28-80	08-15-80	EXCEEDED EFFLUENT TOTAL DISSOLVED SOLIDS LIMIT.
80-38/ 04L-0	07-29-80	08-14-80	EXCEEDED EFFLUENT CHLORINE LIMIT.

REVIEWED BY OMPA LOEB: John L. Crooks **DATE:** 10/6/80

REVIEWED BY NRR DL: Daniel J. Garner **DATE:** 10/8/80

RANCHO SECO

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-312
2. Utility Contact: R. W. Colombo (916) 452-3211
3. Licensed Thermal Power (MWt): 2772
4. Nameplate Rating (Gross MWe): 1070 x 0.9 = 963
5. Design Electrical Rating (Net MWe): 918
6. Maximum Dependable Capacity (Gross MWe): 917
7. Maximum Dependable Capacity (Net MWe): 873
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	5,855.0	47,112.0
12. Hours Reactor Was Critical	290.5	2,548.5	28,622.0
13. Reactor Reserve Shutdown Hours	.0	.0	3,975.1
14. Hours Generator On-Line	271.8	2,445.4	27,386.7
15. Unit Reserve Shutdown Hours	.0	.0	1,213.2
16. Gross Thermal Energy (MWH)	734,688	6,281,304	69,315,002
17. Gross Electrical Energy (MWH)	243,168	2,096,300	23,380,836
18. Net Electrical Energy (MWH)	226,112	1,973,909	22,084,799
19. Unit Service Factor	36.5	41.8	58.1
20. Unit Availability Factor	36.5	41.8	60.7
21. Unit Capacity Factor (MDC Net)	34.8	38.6	53.7
22. Unit Capacity Factor (DER Net)	33.1	36.7	51.1
23. Unit Forced Outage Rate	63.5	18.4	31.1

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: NO ESTIMATE

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
7	08/12/80	F	472.2	A	3		HA		TURBIN DAMAGED THRUST BEARING.

SUMMARY

RANCHO SECO REMAINED ON LINE UNTIL AUGUST 12TH, WHEN AN OUTAGE OCCURRED DUE TO A DAMAGED THRUST BEARING.

LEGEND

[1] Type:
F–Forced
S–Scheduled

[2] Reason:
A–Equipment Failure
B–Maintenance or Test
C–Refueling
D–Regulatory Restriction
E–Operator Training & License Examination
F–Administrative
G–Operational Error
H–Other

[3] Method:
1–Manual
2–Manual Scram
3–Automatic Scram
4–Continuations
5–Load Reduction
9–Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

NRC POWER RESTRICTIONS: NONE

ROBINSON 2

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	South Carolina/Darlington 5miles NW of Hartsville
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2300
5.	Design Electrical Rating (MWE Net)	700
6.	Date of Initial Criticality	September 20, 1970
7.	Date of Commercial Operation	March 7, 1971
8.	Condenser Cooling Method	Recirculation from impounded lake
9.	Condenser Cooling Water Source	Robinson Impoundment

Utility & Contractor Information

10.	Licensee	Carolina Power and Light Company
11.	Corporate Address	411 Fayetteville Street Raleigh, North Carolina
12.	Corporate Contact	J. A. Jones Executive Vice President
13.	Architect/Engineer	Ebasco
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Ebasco

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R.C. Lewis
18.	IE Section Chief	P. J. Kellogg
19.	IE Resident Inspector	S. Weiss
20.	Licensing Project Manager	D. Neighbors
21.	Docket Number	50-261
22.	License Number/Date of Issue	DPR-23, 7/31/70 (5 Mwt), Full power authorization issued 9/23/70
23.	Public Document Room	Hartsville Memorial Library Home and Fifth Avenue Hartsville, South Carolina 29550

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

SHUTDOWN FOR REFUELING, AUGUST 1980.

LAST IE SITE INSPECTION DATE: JULY 18, 1980

INSPECTION REPORT NO: 50-261/80-16

SUMMARY

INSPECTION SUMMARY

INSPECTION JUNE 24 - JULY 2 (80-15): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 83 INSPECTOR-HOURS ON SITE IN THE AREAS OF WITNESSING GENERAL PLANT OPERATIONS, FOLLOWUP INSPECTION OF OUTSTANDING ITEMS, REVIEW OF PLANT SAFETY COMMITTEE MEETING MINUTES, AND REVIEW OF CONTAINMENT PURGE VALVE ISOLATION SYSTEM. OF THE FOUR AREAS INSPECTED, NO ITEMS 0 NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION JULY 7-18 (80-16): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 72 INSPECTOR-HOURS ON SITE IN THE AREAS OF OPERATIONS, FOLLOWUP OF LICENSEE EVENT REPORTS AND BULLETINS. OF THE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

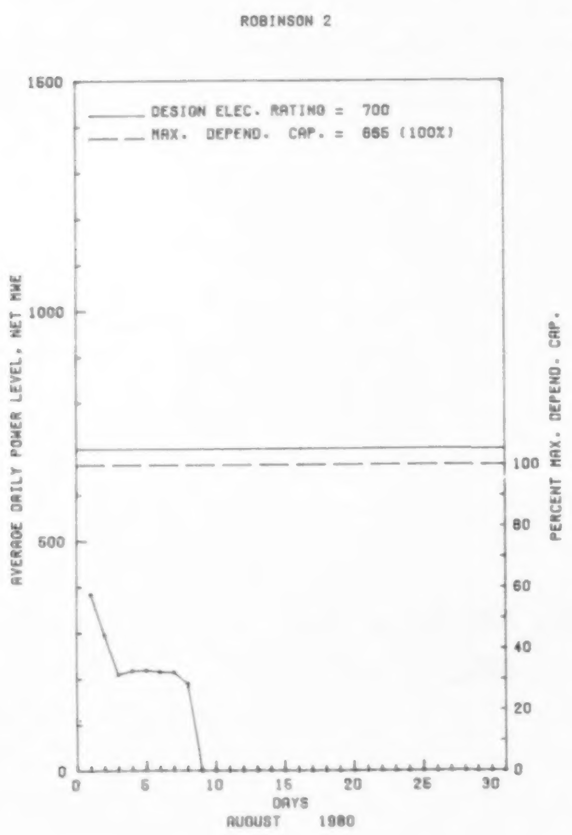
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-014/ 01T-0	07/02/80	07/16/80	NO CERTIFICATIONS AVAILABLE FOR ELECTRICAL CABLE PIGTAILS ON EIGHT PENETRATIONS
80-015/ 01T-0	07/26/80	08/07/80	FAILURE OF NFBD RELAYS
80-016/ 03L-0	07/10/80	08/21/80	TWO GRINELL SNUBBERS INOPERABLE

REVIEWED BY OMPA LOEB: T. C. Cintula **DATE:** 10·6·80

REVIEWED BY NRR DL: Don Neighbors **DATE:** 10/7/80

ROBINSON 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-261
2. Utility Contact: M. L. Watford (803) 383-4524
3. Licensed Thermal Power (MWt): 2300
4. Nameplate Rating (Gross MWe): 854 x 0.9 = 769
5. Design Electrical Rating (Net MWe): 700
6. Maximum Dependable Capacity (Gross MWe): 700
7. Maximum Dependable Capacity (Net MWe): 665
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): 2200 MWt
10. Reasons for Restrictions, If Any: EXCESS MOISTURE CARRYOVER TO H.P. TURBINE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	5,855.0	83,237.0
12. Hours Reactor Was Critical	191.2	4,306.7	64,513.6
13. Reactor Reserve Shutdown Hours	.0	219.8	939.1
14. Hours Generator On-Line	190.2	4,282.6	62,973.5
15. Unit Reserve Shutdown Hours	.0	.0	23.2
16. Gross Thermal Energy (MWH)	193,421	8,207,853	128,758,142
17. Gross Electrical Energy (MWH)	52,329	2,605,015	41,541,528
18. Net Electrical Energy (MWH)	47,117	2,454,097	39,359,199
19. Unit Service Factor	25.6	73.1	75.7
20. Unit Availability Factor	25.6	73.1	75.7
21. Unit Capacity Factor (MDC Net)	9.5	63.0	71.1
22. Unit Capacity Factor (DER Net)	9.0	59.9	67.6
23. Unit Forced Outage Rate	.0	19.2	13.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 10/02/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
08-01	08/08/80	S	553.8	C	1		RC	FUELXX	REFUELING OUTAGE & MAINTENANCE.

SUMMARY

ROBINSON 2 REMAINED ON LINE UNTIL AUGUST 8TH, WHEN IT COMMENCED A REFUELING AND MAINTENANCE OUTAGE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source

SALEM 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	New Jersey/Salem 20 miles S of Wilmington, Del.
2.	Electric Reliability Council	Mid-Atlantic Area Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	3338
5.	Design Electrical Rating (MWE Net)	1090
6.	Date of Initial Criticality	December 11, 1976
7.	Date of Commercial Operation	June 30, 1977
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Delaware River

Utility & Contractor Information

10.	Licensee	Public Service Electric & Gas Co.
11.	Corporate Address	80 Park Place - 15A Newark, New Jersey 07101
12.	Corporate Contact	F. P. Librizzi General Manager - Electric Prod.
13.	Architect/Engineer	PSE&G
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	UE&C

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	IE Section Chief	R. R. Keimig
19.	IE Resident Inspector	L. J. Norrholm (609-935-3850) (FTS 341-3000)
20.	Licensing Project Manager	W. Ross
21.	Docket Number	50-272
22.	License Number/Date of Issue	DPR-70, 8/13/76
23.	Public Document Room	Salem Free Public Library 112 West Broadway Salem, New Jersey 08079

INSPECTION STATUS

ENFORCEMENT STATUS

FACILITY DESIGN CHANGE REVIEW PER 10 CFR 55, APP. A, 3. ON-THE-JOB TRAINING NOT CONDUCTED BY ALL LICENSED OPERATORS AND SENIOR OPERATORS. (8014 INFRACTION)

DOCUMENTATION OF SIMULATOR TRAINING REQUIRED BY TRAINING PROCEDURE TO IMPLEMENT 10 CFR 55, APP. A, 5. RECORDS WERE NOT IN INDIVIDUALS TRAINING FOLDERS.
(8014 DEFICIENCY)

FACILITY DESIGN CHANGE REVIEW PER 10 CFR 55, APP. A, 3. ON-THE-JOB TRAINING NOT CONDUCTED BY ALL LICENSED OPERATORS AND SENIOR OPERATORS. (8014 INFRACTION)

DOCUMENTATION OF SIMULATOR TRAINING REQUIRED BY TRAINING PROCEDURE TO IMPLEMENT 10 CFR 55, APP. A, 5. RECORDS WERE NOT IN INDIVIDUALS TRAINING FOLDERS.
(8014 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

FACILITY ITEMS (PLANS AND PROCEDURES):

THE SECOND REFUELING OUTAGE IS SCHEDULED TO BEGIN SEPTEMBER 20, 1980.

MANAGERIAL ITEMS:

THE LICENSEE ANNOUNCED THE FOLLOWING MANAGEMENT CHANGES, EFFECTIVE 9/1/80:

-- JOHN ZUPKO, CHIEF ENGINEER (SALEM) TO MANAGER - NUCLEAR OPERATIONS (CORPORATE OFFICE)

-- JOHN DRISCOLL, STATION OPERATING ENGINEER (SALEM) TO CHIEF ENGINEER (SALEM)

-- L. M. FRY, STATION OPERATING ENGINEER (HOPE CREEK) TO STATION OPERATING ENGINEER (SALEM)

PLANT STATUS:

UNIT IS IN A POWER COAST DOWN, PRESENTLY AT 65%.

LAST IE SITE INSPECTION DATE: 8/1-31/80

INSPECTION REPORT NO: 50-272/80-20

SUMMARY

INSPECTION SUMMARY

50-272/80-16 - JUL 1-31: ROUTINE INSPECTION BY TWO RESIDENT INSPECTORS (72 HRS) OF PLANT OPERATIONS INCLUDING TOURS OF THE FACILITY; LOG AND RECORD REVIEWS; REVIEW OF LICENSEE EVENTS; OPERATIONAL SAFETY VERIFICATION; IMPLEMENTATION AND TERMINATION OF STRIKE CONTINGENCY PLANS; AND TRAINING STAFF QUALIFICATIONS. NO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED.

50-272/80-17 - JUL 14-18: ROUTINE, UNANNOUNCED PHYSICAL SECURITY INSPECTION BY ONE REGION-BASED INSPECTOR (39 HRS) OF THE APPROVED PHYSICAL SECURITY PLAN, SECURITY PROGRAM AUDIT, SECURITY ORGANIZATION, PHYSICAL BARRIERS (PA/VA), LIGHTING, ACCESS CONTROL (SEARCH) (ESCORTING), (VITAL AREAS), ALARM STATION, DETECTION AIDS, ASSESSMENT AIDS, COMMUNICATIONS, TESTING AND MAINTENANCE, COMPENSATORY MEASURES, POWER SUPPLY, RESPONSE, LOCKS, KEYS AND COMBINATIONS AND RECORDS AND REPORTS. THREE ITEMS OF NONCOMPLIANCE WERE IDENTIFIED: FAILURE TO KEEP THE ISOLATION ZONE CLEAR OF STRUCTURES OR MATERIAL; FAILURE TO SECURE AN OPENING IN THE WALL OF THE CONTROL AND AUXILIARY RELAY ROOM; FAILURE TO SECURE AN OPENING ABOVE A PARTITION THAT WAS PART OF A VITAL AREA BARRIER.

50-272/80-18 - JUL 16-18: ROUTINE, ANNOUNCED INSPECTION BY ONE REGION-BASED INSPECTOR (18 HRS) OF QUALIFICATION DATA FOR THE ELECTRICAL EQUIPMENT INSIDE CONTAINMENT ASSOCIATED WITH THE ELECTRICAL SYSTEM, ACCUMULATOR SYSTEM, AND THE STEAM GENERATOR SYSTEM AS DEFINED IN THE LICENSEE'S RESPONSE TO IE BULLETIN 79-01B DATED APRIL 18, 1980. NO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-038/ 03L	06/26/80	07/23/80	12 CHARGING PUMP INOPERABLE-LUBE OIL COOLER FAILURE AND REPLACEMENT.
80-039/ 03L	07/02/80	07/23/80	12 CONTAINMENT FAN COIL UNIT INOPERABLE - SOLENOID VALVE 624 FAILURE.
80-040/ 01T	07/2[illegible]/80	08/08/80	LOSS OF OFF SITE COMMUNICATIONS.
80-041/ 03L	07/08/80	08/04/80	NO. 11 SAFETY INJECTION PUMP INOPERABLE - LUBE OIL COOLER TUBE FAILURE.
80-042/ 03L	07/21/80	08/19/80	24V BATTERY SURVEILLANCE NOT PERFORMED WITHIN REQUIRED TIME INTERVAL.

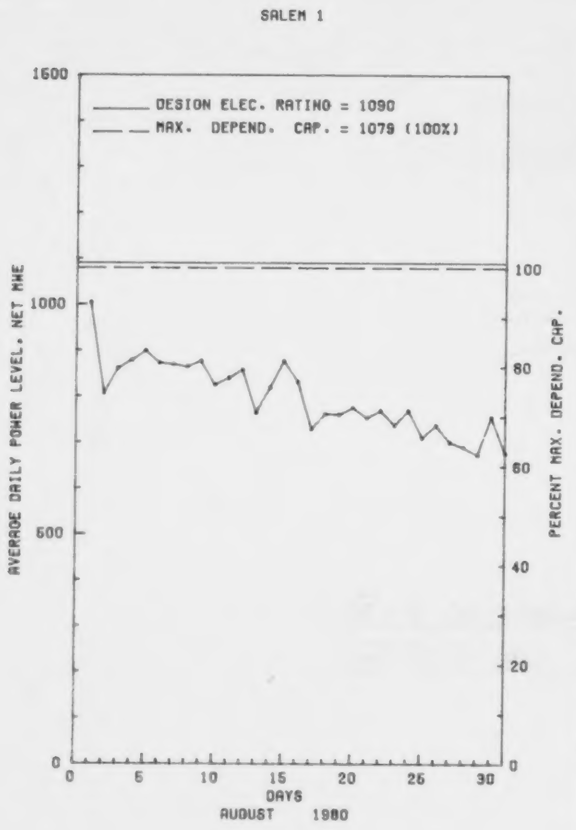
REVIEWED BY OMPA LOEB: T. C. Cintula **DATE:** 10·6·80

REVIEWED BY NRR DL: W J Ross **DATE:** 10/7/80

best copy available

SALEM 1

AVERAGE DAILY POWER LEVEL(MWe)



OPERATING STATUS

1. Docket Number: 50-272
2. Utility Contact: L. K. Miller (609) 365-7000
3. Licensed Thermal Power (MWt): 3338
4. Nameplate Rating (Gross MWe): 1300 x 0.9 = 1170
5. Design Electrical Rating (Net MWe): 1090
6. Maximum Dependable Capacity (Gross MWe): 1124
7. Maximum Dependable Capacity (Net MWe): 1079
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	5,855.0	27,816.0
12. Hours Reactor Was Critical	744.0	5,665.9	15,754.2
13. Reactor Reserve Shutdown Hours	.0	.0	22.7
14. Hours Generator On-Line	744.0	5,546.0	15,077.1
15. Unit Reserve Shutdown Hours	0	.0	.0
16. Gross Thermal Energy (MWH)	2,440,018	17,414,364	44,950,660
17. Gross Electrical Energy (MWH)	623,120	5,698,710	14,866,210
18. Net Electrical Energy (MWH)	590,923	5,450,636	14,080,226
19. Unit Service Factor	100.0	94.7	54.2
20. Unit Availability Factor	100.0	94.7	54.2
21. Unit Capacity Factor (MDC Net)	73.6	86.3	46.9
22. Unit Capacity Factor (DER Net)	72.9	85.4	46.4
23. Unit Forced Outage Rate	.0	5.3	34.8

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUELING - SEPTEMBER 20, 1980.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-158	08/01/80	F	.0	A	5		HH		FILTER CLEAN #11 & #12 HEATER DRAIN PUMP SCREENS.
80-160	08/01/80	S	.0	H	5		RC	ZZZZZZ	FUEL DEPLETION
80-161	08/02/80	S	.0	H	5		RC	ZZZZZZ	FUEL DEPLETJON
80-162	08/02/80	F	.0	A	5		HH		FILTER CLEAN #11 & #12 & #13 CONDENSATE PUMP SUCTION STRAINERS.
80-163	08/02/80	F	.0	A	5		RB		CONROD ROD CONTROL MALFUNCTION.
80-164	08/03/80	S	.0	H	5		RC	ZZZZZZ	FUEL DEPLETION
80-165	08/06/80	S	.0	H	5		RC	ZZZZZZ	FUEL DEPLETION
80-166	08/08/80	S	.0	H	5		RC	ZZZZZZ	FUEL DEPLETION
80-168	08/10/80	S	.0	H	5		RC	ZZZZZZ	FUEL DEPLETION
80-170	08/12/80	S	.0	H	5		RC	ZZZZZZ	FUEL DEPLETION
80-172	08/16/80	S	.0	H	5		RC	ZZZZZZ	FUEL DEPLETION

SUMMARY

SALEM 1 REMAINED ON-LINE FOR THE ENTIREMONTH, WITH NUMEROUS REDUCTIONS LISTED IN DETAIL ABOVE. UNIT IS OPERATING AT END-OF CYCLE COASTDOWN IN PREPARATION FOR REFUELING.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

SAN ONOFRE 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	California/San Diego 5 miles S of San Clemente
2.	Electric Reliability Council	Western Systems Coordinating Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	1347
5.	Design Electrical Rating (MWE Net)	436
6.	Date of Initial Criticality	June 14, 1967
7.	Date of Commercial Operation	January 1, 1968
8.	Condenser Cooling Method	Circulating sea water
9.	Condenser Cooling Water Source	Pacific Ocean

Utility & Contractor Information

10.	Licensee	Southern California Edison Co.
11.	Corporate Address	2244 Walnut Grove Avenue Rosemead, California 91770
12.	Corporate Contact	R. E. Woodbury V.P. & General Counsel
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	V
17.	IE Branch Chief	J. L. Crews
18.	IE Section Chief	B. H. Faulkenberry
19.	IE Resident Inspector	L. F. MILLER (714-492-2641)
20.	Licensing Project Manager	S. J. Nowicki
21.	Docket Number	50-206
22.	License Number/Date of Issue	DPR-13, 3/27/67
23.	Public Document Room	Mission Viejo Branch Library 24851 Chrisanta Drive Mission Viejo, California 92676

INSPECTION STATUS

ENFORCEMENT STATUS

TITLE 10, CFR 20.203(F)(1)&(2) REQUIFES THAT CONTAINERS OF LICENSED MATERIALS BEAR A DURABLE CLEARLY VISABLE LABEL IDENTIFYING THE RADIOACTIVE CONTENTS. SECTION (F)(3) OF THIS REQUIREMENT PROVIDES CERTAIN EXCLUSIONS TO THIS REQUIREMENT. CONTRARY TO THE REQUIREMENTS, ON MAY 19, 1980, MULTIPLE CONTAINERS OF PACKAGED OR PARTIALLY PACKAGED RADIOACTIVE WASTE WERE OBSERVED BY THE INSPECTOR TO BE UNLABELED. APPROPRIATE LABELING WAS PROMPTLY APPLIED WHEN CALLED TO THE LICENSEE'S ATTENTION, AN ACTION WHICH WAS CONFIRMED BY THE INSPECTOR ON MAY 20, 1980. NONE OF THE EXCLUSIONS IN 10 CFR 20.203(F)(3) WERE APPLICABLE. (8017 DEFICIENCY)

INSPECTION STATUS

SYSTEMS AND COMPONENT PROBLEMS:

REFUELING OUTAGE COMMENCED TWO DAYS EARLIER THAN SCHEDULED DUE TO INCREASE IN PRIMARY TO SECONDARY (S/G TUBE) LEAKAGE (PNO-V-80-24).

LARGE NUMBER OF TUBES DETERMINED TO NEED PLUGGING (RESULTS OF EDDY CURRENT TESTING).

FACILITY ITEMS (PLANS AND PROCEDURES):

REFUELING; STEAM GENFRATOR REPAIRS.

MANAGERIAL ITEMS

NONE.

PLANT STATUS:

PLANT SHUTDOWN FOR REFUELING OUTAGE AND STEAM GENERATOR REPAIRS.

LAST IE SITE INSPECTION DATE: 08/18-22/80+

INSPECTION REPORT NO: 50-206/80-23+

SUMMARY

INSPECTION SUMMARY

INSPECTION ON MAY 19-30, 1980 (REPORT NO. 50-206/80-17) AREAS INSPECTED: SPECIAL, ANNOUNCED APPRAISAL OF HEALTH PHYSICS PROGRAM, INCLUDING ORGANIZATION AND MANAGEMENT; QUALIFICATIONS AND TRAINING; QUALITY ASSURANCE; PROCEDURES; EXTERNAL AND INTERNAL EXPOSURE CONTROLS; SURVEYS AND ACCESS CONTROLS; INSTRUMENTATION; ALARA; RADIOACTIVE WASTE; FACILITIES AND EQUIPMENT AND ACCIDENT RESPONSE CAPABILITIES. THE INSPECTION INVOLVED 350 INSPECTOR HOURS ONSITE BY FOUR NRC INSPECTORS.

RESULTS: SEVERAL SIGNIFICANT WEAKNESSES IN THE HEALTH PHYSICS PROGRAM WERE IDENTIFIED. THESE WEAKNESSES ARE IN THE AREAS OF STAFFING, TRAINING, PORTABLE RADIATION PROTECTION INSTRUMENTS, RADIOACTIVE WASTE MANAGEMENT, FACILITIES, AND EMERGENCY RESPONSE CAPABILITIES. ONE ITEM OF NONCOMPLIANCE WAS IDENTIFIED (DEFICIENCY - LABELING RADIOACTIVE WASTE CONTAINERS).

INSPECTION ON JUNE 30 - JULY 24, 1980 (REPORT NO. 50-206/80-21) AREAS INSPECTED: ROUTINE, RESIDENT INSPECTION OF PLANT OPERATIONS DURING LONG TERM OUTAGE; MONTHLY MAINTENANCE AND SURVEILLANCE OBSERVATIONS; AND INDEPENDENT INSPECTION. THE INSPECTION INVOLVED 64 INSPECTOR HOURS ONSITE BY ONE NRC INSPECTOR.

RESULTS: THREE ITEMS OF NONCOMPLIANCE WERE IDENTIFIED (INFRACTION AND DEFICIENCY - FAILURE TO FOLLOW RADIATION EXPOSURE CONTROL PROCEDURE) AND (DEFICIENCY - FAILURE TO IMPLEMENT A PHYSICAL SECURITY PLAN REQUIREMENT).

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-24/ 01T-1	05-16-80	07-16-80	FINAL REPORT ON MAIN STEAM PIPING WELD RADIOGRAPHIC INDICATIONS.
80-31/ 03L-0	07-28-80	08-19-80	SALT WATER PUMP DISCHARGE VALVE POV-6 FAILED TO OPEN-WRONG SPRING IN SOLENOID.
80-32/ 03L-0	07-17-80	08-18-80	CV-537 FAILED TO CLOSE DESICCANT IN SOLENOID.

INSPECTION ON JULY 8-10, 1980 (REPORT NO. 50-206/80-22) A SPECIAL INSPECTION WAS CONDUCTED TO DETERMINE THE CIRCUMSTANCES AND EVENTS THAT RESULTED IN THE APPARENT EXPOSURE OF AN INDIVIDUAL EMPLOYED AT THE SAN ONOFRE UNIT 1 SITE IN EXCESS OF THE LIMITS OF 10 CFR 20.101(B)(1). PERTINENT RECORDS WERE EXAMINED, AND LICENSEE REPRESENTATIVES WERE INTERVIEWED WITH REGARD TO THE ACTIVITIES AND CIRCUMSTANCES THAT LED TO THE DETERMINATION OF THE APPARENT EXPOSURE OF THE INDIVIDUAL IN EXCESS OF REGULATORY LIMITS. THE INSPECTION INVOLVED 47 INSPECTOR HOURS ONSITE BY TWO NRC INSPECTORS.

RESULTS: ONE ITEM OF NONCOMPLIANCE WITH THE SAN ONOFRE UNIT 1 TECHNICAL SPECIFICATION 6.11, AND TWO ITEMS OF NONCOMPLIANCE WITH 10 CFR 20 (20.101(B)(1) AND 20.201(B)) WERE IDENTIFIED.

INSPECTION ON AUGUST 18-22, 1980 (REPORT NO. 50-206/80-23) REPORT BEING PREPARED; TO BE REPORTED NEXT MONTH.

INSPECTION ON AUGUST 4-8, 1980 (REPORT NO. 50-206/80-24) AREAS INSPECTED: ROUTINE, ANNOUNCED INSPECTION OF FIRE PROTECTION MODIFICATIONS, IE BULLETIN/CIRCULAR FOLLOWUP, AND INDEPENDENT INSPECTION EFFORT. THE INSPECTION INVOLVFD 25 INSPECTOR HOURS ONSITE BY ONE NRC INSPECTOR.

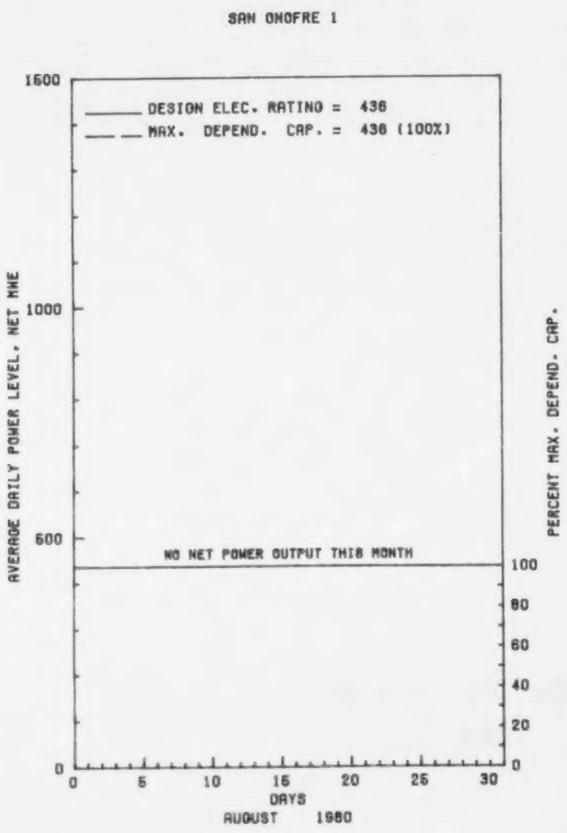
RESULTS: NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

REVIEWED BY OMPA LOEB: T. C. Cintula **DATE:** 10·6·80

REVIEWED BY NRR DL: S. J. Nowicki **DATE:** 10/8/80

SAN ONOFRE 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-206
2. Utility Contact: J. M. JOY (714) 492-7700
3. Licensed Thermal Power (MWt): 1347
4. Nameplate Rating (Gross MWe): 500 x 0.9 = 450
5. Design Electrical Rating (Net MWe): 436
6. Maximum Dependable Capacity (Gross MWe): 456
7. Maximum Dependable Capacity (Net MWe): 436

CAPACITY FACTORS CALCULATED WITH A WEIGHTED AVERAGE

8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	5,855.0	115,831.0
12. Hours Reactor Was Critical	.0	1,979.1	84,611.3
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	.0	1,961.6	81,114.0
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	2,552,787	104,084,261
17. Gross Electrical Energy (MWH)	0	855,000	35,526,434
18. Net Electrical Energy (MWH)	0	816,676	33,664,042
19. Unit Service Factor	.0	33.5	70.0
20. Unit Availability Factor	.0	33.5	70.0
*21. Unit Capacity Factor (MDC Net)	.0	32.0	71.5
22. Unit Capacity Factor (DER Net)	.0	32.0	71.5
23. Unit Forced Outage Rate	100.0	39.5	10.6

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 11/01/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
55	07/12/80	F	744.0	B	4	80-014	CC	HTEXCH	STEAM GENERATOR TUBE REPAIR CONTINUES.

SUMMARY

SAN ONOFRE REMAINS SHUTDOWN TO REPAIR STEAM GENERATOR TUBES.

LEGEND

[1] Type:
- F-Forced
- S-Scheduled

[2] Reason:
- A-Equipment Failure
- B-Maintenance or Test
- C-Refueling
- D-Regulatory Restriction
- E-Operator Training & License Examination
- F-Administrative
- G-Operational Error
- H-Other

[3] Method:
- 1-Manual
- 2-Manual Scram
- 3-Automatic Scram
- 4-Continuations
- 5-Load Reduction
- 9-Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

ST. LUCIE 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Florida/St. Lucie 12 miles SE of Ft. Pierce
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2560
5.	Design Electrical Rating (MWE Net)	802
6.	Date of Initial Criticality	April 22, 1976
7.	Date of Commercial Operation	December 21, 1976
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Atlantic Ocean

Utility & Contractor Information

10.	Licensee	Florida Power & Light Co.
11.	Corporate Address	9250 W. Flagler Street P. O. Box 529100 Miami, Florida 33152
12.	Corporate Contact	R. E. Uhrig, V.P.
13.	Architect/Engineer	Ebasco
14.	Nuclear Steam Supply System	Combustion Engineering
15.	Constructor	Ebasco

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. LEWIS
18.	IE Section Chief	R. A. Martin
19.	IE Resident Inspector	S. A. ELROD
20.	Licensing Project Manager	C. Nelson
21.	Docket Number	50-335
22.	License Number/Date of Issue	DPR-67, March 1, 1976
23.	Public Document Room	Indian River Junior College Library 3209 Virginia Avenue Ft. Pierce, Florida 33450

INSPECTION STATUS

ENFORCEMENT STATUS

SURVEILLANCE INSPECTION PROCEDURE FOR FIRE PROTECTION HOSE SYSTEMS WAS NOT REVIEWED BY THE PLANT FACILITY REVIEW GROUP AND APPROVED BY PLANT MANAGER AS REQUIRED BY SECTION 6.8.2 OF THE TECHNICAL SPECIFICATIONS. (8021 INFRACTION)

THE LICENSEE'S FIRE PROTECTION ADMINISTRATIVE CONTROL PROCEDURES DO NOT FULLY MEET THE NRC GUIDELINES OF THE DOCUMENT ENTITLED "NUCLEAR PLANT FIRE PROTECTION FUNCTIONAL RESPONSIBILITIES, ADMINISTRATIVE CONTROLS AND QUALITY ASSURANCE AS COMMITTED TO BY THE LICENSEE IN LETTER L-78-280 FROM R. E. UHRIG OF FP&L TO V. STELLO OF NRC DATED AUGUST 28, 1978.
(8021 DEVIATION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

NORMAL OPERATION.

LAST IE SITE INSPECTION DATE: JULY 23-31, 1980

INSPECTION REPORT NO: 50-335/80-27

SUMMARY

INSPECTION SUMMARY

INSPECTION JUNE 9-19 (80-17): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 91 INSPECTOR-HOURS ON SITE IN THE AREAS OF BULLETIN AND CIRCULAR REVIEW; FOLLOW UP OF REACTOR TRIP AND NATURAL CIRCULATION COOLDOWN OCCURRING JUNE 11, 1980; THREE MILE ISLAND LESSONS LEARNED SHORT-TERM CORRECTIVE ACTIONS; PLANT CHANGE TO THE AUXILIARY FEEDWATER SYSTEM. OF THE FOUR AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN THREE AREAS; ONE ITEM OF NONCOMPLIANCE WAS FOUND IN ONE AREA - (INFRACTION - FAILURE TO PROMPTLY NOTIFY NRC VIA THE EMERGENCY NOTIFICATION TELEPHONE NETWORK WHEN THE UNIT RESPONDED IN AN UNEXPECTED MANNER).

INSPECTION JULY 8-11 (80-21): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 26 INSPECTOR-HOURS ON SITE IN THE AREAS OF FIRE PROTECTION/PREVENTION. OF THE AREAS INSPECTED, ONE ITEM OF NONCOMPLIANCE WAS FOUND (INFRACTION - INADEQUATE REVIEW AND APPROVAL OF FIRE PROTECTION HOSE SYSTEMS SURVEILLANCE INSPECTION PROCEDURES); AND, ONE APPARENT DEVIATION WAS FOUND (DEVIATION - FIRE PROTECTION ADMINISTRATIVE PROCEDURES DO NOT MEET THE NRC GUIDELINES).

INSPECTION JULY 8-17 (80-22): THIS ANNOUNCED INSPECTION INVOLVED 45 INSPECTOR-HOURS ON SITE IN THE AREAS OF BULLETIN, CIRCULAR, AND INFORMATION NOTICE REVIEW; LICENSEE ACTIONS WITH RESPECT TO NRC HEADQUARTERS CORRESPONDENCE; PLANT OPERATIONS. OF THE THREE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION JULY 15-18 (80-23): THIS SPECIAL, UNANNOUNCED INSPECTION INVOLVED 25 INSPECTOR-HOURS ON SITE IN THE AREAS OF REVIEW OF RECORDS ASSOCIATED WITH WORK PERFORMED DURING AN OUTAGE IN THE AREAS OF INSTALLATION WELDING, WELD ROD CONTROL, CORE DRILLING, HANGER INSTALLATION, NONCONFORMANCE REPORT CONTROL, AND INSPECTOR QUALIFICATIONS. OF THE EIGHT AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION JUNE 24 - JULY 2 (80-25): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 68 INSPECTOR-HOURS ON SITE. THE AREAS INSPECTED INCLUDED A REVIEW OF POST-REFUELING STARTUP TESTS AND OBSERVATION OF CONTROL ROOM ACTIVITIES. ONE ITEM OF NONCOMPLIANCE WAS IDENTIFIED IN ONE AREA (INFRACTION - FAILURE TO USE THE CURRENT VERSION OF A PROCEDURE).

INSPECTION JULY 23-31 (80-27): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 61 RESIDENT INSPECTOR-HOURS ON SITE IN THE AREAS OF PLANT OPERATIONS, PLANT TOUR, FOLLOWUP OF IE BULLETIN 79-15, AND FOLLOWUP OF HEADQUARTERS' CONCERNS. OF THE FOUR AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

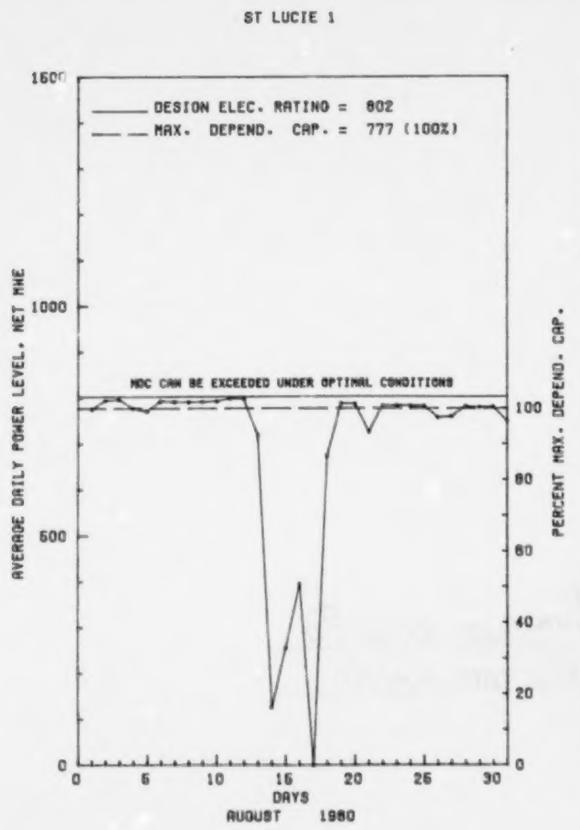
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-026/03L	05/30/80	06/29/80	31-DAY FIRE STATION INSPECTION INTERVAL EXCEEDED
80-028/03L	05/31/80	06/30/80	CHANNEL A LOW PRESSURE S/G TRIP FAILED DUE TO LOSS OF POWER SUPPLY
80-031/03L	07/03/80	08/04/80	CALIBRATION OF DT POWER CHANNEL WAS MISSED JULY 3, 1980
80-032/03L	07/16/80	08/13/80	CEA DROPPED TO FULL INSERTION

REVIEWED BY OMPA LOEB: MR Beebe **DATE:** 10/06/80

REVIEWED BY NRR DL: C Nelson **DATE:** 10-7-80

ST. LUCIE 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-335
2. Utility Contact: V. T. Chilson (305) 552-3824
3. Licensed Thermal Power (MWt): 2560
4. Nameplate Rating (Gross MWe): 1000 x 0.85 = 850
5. Design Electrical Rating (Net MWe): 802
6. Maximum Dependable Capacity (Gross MWe): 822
7. Maximum Dependable Capacity (Net MWe): 777
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	5,855.0	32,399.0
12. Hours Reactor Was Critical	720.0	4,000.2	25,510.1
13. Reactor Reserve Shutdown Hours	.0	.0	129.4
14. Hours Generator On-Line	701.4	3,889.8	24,719.9
15. Unit Reserve Shutdown Hours	7.3	7.3	39.3
16. Gross Thermal Energy (MWH)	1,697,623	9,575,354	59,834,520
17. Gross Electrical Energy (MWH)	551,240	3,101,490	19,395,990
18. Net Electrical Energy (MWH)	519,281	2,915,036	18,242,932
19. Unit Service Factor	94.3	66.4	76.3
20. Unit Availability Factor	95.3	66.6	76.4
21. Unit Capacity Factor (MDC Net)	89.8	64.1	72.3
22. Unit Capacity Factor (DER Net)	87.0	62.1	70.2
23. Unit Forced Outage Rate	.0	11.1	6.3

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
11	08/14/80	S	10.7	A	1		EG	TRANSF	UNIT WAS REMOVED FROM SERVICE TO ISOLATE MAIN POWER TRANSFORMER NO. 1B TO REPAIR OIL LEAK. CORRECTIVE ACTIONS INCLUDED INSTALLING NEW BUSHING.
12	08/14/80	S	.0	B	5		EG	TRANSF	LOAD WAS LIMITED TO THE CAPACITY OF MAIN TRANSFORMER NO. 1A.
13	08/14/80	S	7.3	F	1		EG	TRANSF	UNIT WAS REMOVED FROM SERVICE TO PROTECT PERSONNEL USING A CRANE TO REMOVE A FAULTY BUSHING FROM MAIN POWER TRANSFORMER NO. 1B.
14	08/17/80	S	24.6	B	1		EG	TRANSF	UNIT WAS REMOVED FROM SERVICE TO INSTALL NEW BUSHING AND RETURN MAIN POWER TRANSFORMER NO. 1B TO SERVICE.

SUMMARY

ST. LUCIE OPERATED NORMALLY, WITH ONE REDUCTION AND THREE OUTAGES TO PERFORM MAINTENANCE CONCERNING TRANSFORMER 1B.

LEGEND

[1] Type:
- F–Forced
- S–Scheduled

[2] Reason:
- A–Equipment Failure
- B–Maintenance or Test
- C–Refueling
- D–Regulatory Restriction
- E–Operator Training & License Examination
- F–Administrative
- G–Operational Error
- H–Other

[3] Method:
- 1–Manual
- 2–Manual Scram
- 3–Automatic Scram
- 4–Continuations
- 5–Load Reduction
- 9–Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

SURRY 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Virginia/Surry 17 miles NW of Newport News
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2441
5.	Design Electrical Rating (MWE Net)	822
6.	Date of Initial Criticality	July 1, 1972
7.	Date of Commercial Operation	December 22, 1972
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	James River

Utility & Contractor Information

10.	Licensee	Virginia Electric and Power Company
11.	Corporate Address	P. O. Box 26666 Richmond, Virginia 23261
12.	Corporate Contact	J. H. Ferguson Exec. V.P. - Power
13.	Architect/Engineer	Stone & Webster
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R.C. Lewis
18.	IE Section Chief	P. J. Kellogg
19.	IE Resident Inspector	D. Burke (804) 357-2101
20.	Licensing Project Manager	D. Neighbors
21.	Docket Number	50-280
22.	License Number/Date of Issue	DPR-32, 5/25/72
23.	Public Document Room	Swem Library College of William and Mary Williamsburg, Virginia 23185

INSPECTION STATUS

ENFORCEMENT STATUS

AS REQUIRED BY TECHNICAL SPECIFICATION 6.4.A.7 AND 6.4.D, THE PREVENTATIVE OR CORRECTIVE MAINTENANCE PROCEDURES WHICH WOULD HAVE AN EFFECT ON THE SAFETY OF THE REACTOR MUST BE FOLLOWED. CONTRARY TO THE ABOVE, DURING UNIT 1 OPERATIONS ON JUNE 16, 1980, VARIOUS UNIT 1 SAFETY-RELATED CABLE TRAY COVERS WERE FOUND NOT IN PLACE OR PROPERLY BOLTED, AND RAGS, PAPERS AND PRINTS, TAPE, AND WOOD WERE FOUND ON SEVERAL UNIT 1 SAFETY-RELATED CABLE TRAY COVERS IN THE EMERGENCY SWITCHGEAR ROOM AND CABLE VAULT, INDICATING THAT ELECTRICAL MAINTENANCE PROCEDURES EMP-C-EPH-25 AND -26, AND NUS 9045 WERE NOT FOLLOWED TO ENSURE ADEQUATE FIRE PROTECTION AND ELECTRICAL SEPARATION. (8023 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

STEAM GENERATOR TUBE LEAKS.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

UNIT IS CURRENTLY OPERATING AT 82% AND "COASTING" DOWN.

LAST IE SITE INSPECTION DATE: AUGUST 1, 1980

INSPECTION REPORT NO: 50-280/80-28

SUMMARY

INSPECTION SUMMARY

INSPECTION JUNE 9 - JULY 3 (80-23): THIS ROUTINE INSPECTION BY THE RESIDENT INSPECTOR INVOLVED 45 INSPECTOR-HOURS ON SITE IN THE AREAS OF PLANT OPERATIONS AND OPERATING RECORDS, PLANT MODIFICATIONS, MAINTENANCE AND TESTING, IE BULLETINS, LICENSEE EVENT REPORTS, AND PLANT SECURITY. OF THE FIVE AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE FOUND IN FOUR AREAS; ONE APPARENT ITEM OF NONCOMPLIANCE WAS FOUND IN ONE AREA (INFRACTION - FAILURE TO FOLLOW PLANT MAINTENANCE AND MODIFICATION PROCEDURES).

INSPECTION JULY 9-10 (80-26): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 8 INSPECTOR-HOURS ON SITE IN THE AREAS OF THE IE BULLETIN 79-02, "PIPE SUPPORT BASE PLATE DESIGN USING CONCRETE EXPANSION ANCHOR BOLTS", IE BULLETIN 79-14", SEISMIC ANALYSIS FOR AS-BUILT SAFETY RELATED PIPING SYSTEMS" AND LICENSEE ACTIONS ON PREVIOUS INSPECTION FINDINGS. OF THE THREE AREAS INSPECTED NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION JULY 21-25 (80-27): THIS SPECIAL, ANNOUNCED INSPECTION INVOLVED 31 INSPECTOR-HOURS ON SITE IN THE AREAS OF DOCUMENTATION REVIEW WITH RESPECT TO IE BULLETIN 79-01B RESPONSE. OF THE AREA INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION JULY 29 - AUGUST 1 (80-28): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 13 INSPECTOR-HOURS ON SITE IN THE AREAS OF LICENSEE EVENT REPORTS (LER'S) AND PREVIOUS INSPECTIONS FINDINGS. NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

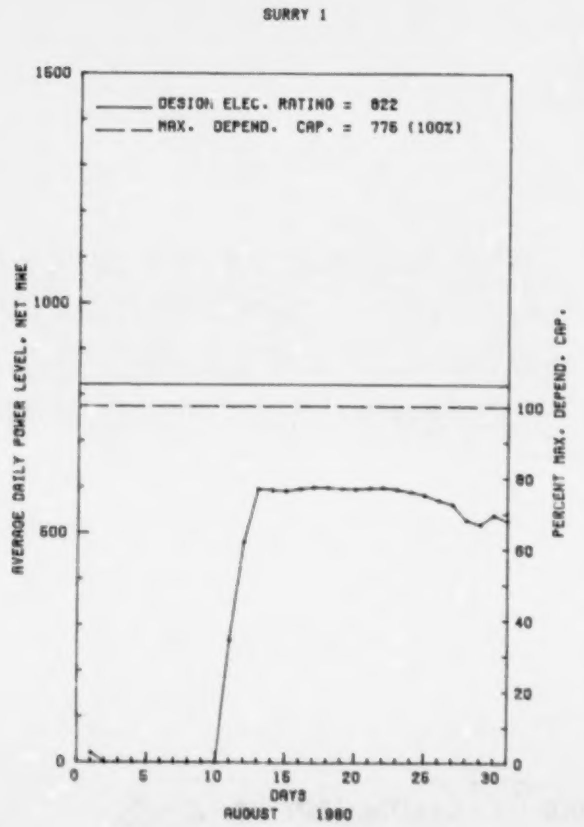
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-037/03L-0	07/11/80	08/06/80	RAD MONITOR ON AIR EJECTOR DISCHARGE OUT OF SERVICE
80-034/03L-0	07/03/80	07/24/80	CONDENSER INLET VALVE INOPERABLE DUE TO BROKEN STUDS ON MOTOR END BELL

REVIEWED BY OMPA LOEB: T. C. Cintula **DATE:** 10·6·80

REVIEWED BY NRR DL: Don Neighbors **DATE:** 10/7/80

SURRY 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-280
2. Utility Contact: SUE DUNN (804) 357-3184
3. Licensed Thermal Power (MWt): 2441
4. Nameplate Rating (Gross MWe): 942 x 0.9 = 848
5. Design Electrical Rating (Net MWe): 822
6. Maximum Dependable Capacity (Gross MWe): 811
7. Maximum Dependable Capacity (Net MWe): 775
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	5,855.0	67,439.0
12. Hours Reactor Was Critical	509.0	3,462.7	42,216.3
13. Reactor Reserve Shutdown Hours	.0	182.6	3,731.5
14. Hours Generator On-Line	503.0	3,440.1	41,345.1
15. Unit Reserve Shutdown Hours	.0	183.6	3,736.2
16. Gross Thermal Energy (MWH)	1,045,464	8,055,870	95,778,752
17. Gross Electrical Energy (MWH)	301,498	2,448,235	31,178,733
18. Net Electrical Energy (MWH)	281,914	2,312,196	29,539,895
19. Unit Service Factor	67.6	58.8	61.3
20. Unit Availability Factor	67.6	61.9	66.8
21. Unit Capacity Factor (MDC Net)	48.9	51.0	56.5
22. Unit Capacity Factor (DER Net)	46.1	48.0	53.3
23. Unit Forced Outage Rate	32.4	41.2	26.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): 9/15/80 - S/G REPLACEMENT OUTAGE; REFUELING - 1 YEAR.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-12	08/01/80	F	241.0	A	1				REACTOR SHUTDOWN DUE TO > .3 GPM PRIMARY TO SECONDARY TUBE LEAK IN "1C" S/G.

SUMMARY

SURRY 1 WAS OFF-LINE FROM THE 1ST THRU THE 10TH FOR A PRIMARY TO SECONDARY TUBE LEAK IN THE 1C STEAM GENERATOR.

LEGEND

[1] Type:
F–Forced
S–Scheduled

[2] Reason:
A–Equipment Failure
B–Maintenance or Test
C–Refueling
D–Regulatory Restriction
E–Operator Training & License Examination
F–Administrative
G–Operational Error
H–Other

[3] Method:
1–Manual
2–Manual Scram
3–Automatic Scram
4–Continuations
5–Load Reduction
9–Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

SURRY 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Virginia/Surry 17 miles NW of Newport News
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2441
5.	Design Electrical Rating (MWE Net)	822
6.	Date of Initial Criticality	March 7, 1973
7.	Date of Commercial Operation	May 1, 1973
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	James River

Utility & Contractor Information

10.	Licensee	Virginia Electric and Power Company
11.	Corporate Address	P. O. Box 26666 Richmond, Virginia 23261
12.	Corporate Contact	J. H. Ferguson Exec. V.P. - Power
13.	Architect/Engineer	Stone & Webster
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R.C. Lewis
18.	IE Section Chief	P. J. Kellog
19.	IE Resident Inspector	D. Burke (804) 357-2101
20.	Licensing Project Manager	D. Neighbors
21.	Docket Number	50-281
22.	License Number/Date of Issue	DPR-37, 1/29/73
23.	Public Document Room	Swem Library College of William and Mary Williamsburg, Virginia 23185

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO 10 CFR 20.201, AT THE TIME OF THE INVESTIGATION (AND ALTHOUGH THE LICENSEE WAS DOCUMENTING THE RELEASES) THE LICENSEE HAD NOT PERFORMED AN EVALUATION OF THE PERIODIC PRESENCE OF TRITIUM, CESIUM-134, CESIUM-137, COBALT-58, COBALT-60 AND IODINE-131 IN WATER ENTERING A TURBINE BUILDING FLOOR DRAIN AND BEING RELEASED TO THE UNRESTRICTED AREA.
(7990 INFRACTION)

CONTRARY TO TS-3.11.A.5, LIQUID WASTE RELEASES (NOS. 79-915 AND 79-916) WERE MADE FROM THE CONTAMINATED DRAIN TANKS ON APRIL 3, 1979, DURING A PERIOD THAT THE LIQUID WASTE DISPOSAL RADIATION MONITOR, RM-LW-108 WAS NOT FUNCTIONING PROPERLY AS INDICATED BY RADIATION RECORDER RR-175. MANY LIQUID WASTE RELEASES WERE MADE FROM THE CONTAMINATED DRAIN TANKS AND THE LIQUID WASTE TEST TANK DURING PERIODS WHEN MAINTENANCE RECORDS (MRS) INDICATED THE DISCHARGE TUNNEL RADIATION MONITOR RM-SW-120 SHOULD HAVE BEEN DECLARED INOPERATIVE. FOR EXAMPLE, MR S1907170621 WHICH WAS INITIATED AT 0621 HOURS ON JULY 17, 1979, DESCRIBED THE PROBLEM AS "MONITOR SPIKES". THE MR INDICATED THAT THE PHOTOMULTIPLIER TUBE IN THE DETECTOR WAS REPLACED AND THE MONITOR WAS RECALIBRATED ON JULY 18, 1979. YET RECORDS INDICATE THAT 10 LIQUID WASTE RELEASES WERE MADE BETWEEN INITIATION OF THE MR AND 2400 HOURS ON JULY 17, 1979. NINE LIQUID WASTE RELEASES WERE MADE DURING THE PERIOD 0600 HOURS TO 2330 HOURS ON FEBRUARY 25, 1979, WHEN DISTANCES BETWEEN TIME MARKS ON THE RADIATION RECORDER RR-175 CHARTS, AS MEASURED BY NRC INVESTIGATORS, INDICATE THAT THE RECORDER WAS WITHOUT CHART PAPER OR WAS TURNED OFF FOR APPROXIMATELY 161 HOURS DURING THAT PERIOD.
(7990 INFRACTION)

CONTRARY TO TS 3.11.A.4, REPRESENTATIVE SAMPLES WERE NOT TAKEN BEFORE RELEASES FROM THE CONTAMINATED DRAIN TANKS ON VARIOUS UNKNOWN DATES IN THAT VARIOUS LICENSEE OPERATORS ADMITTED HAVING: (1) TAKEN SAMPLES WITHOUT HAVING RECIRCULATED THE CONTENTS OF THE TANKS TO ENSURE THOROUGH MIXING; AND (2) TAKEN A SINGLE SAMPLE FROM BOTH TANKS SIMULTANEOUSLY WITHOUT KNOWING THE RELATIVE CONTRIBUTION OF THE INDIVIDUAL TANKS TO THE COMPOSITE SAMPLE.
(7990 INFRACTION)

CONTRARY TO TS 6.4, OPERATING PROCEDURE OP-22.5, "CONTAMINATED DRAINS SYSTEM" AND/OR ANNUNICATOR PROCEDURES 1WD-9 AND 1WD-11, "CONTAMINATED DRAIN TANK HI-LO LEVEL" WERE NOT FOLLOWED ON AT LEAST SIX OCCASIONS DURING 1979 (RELEASE NOS. 79-01, 79-06, 79-545, 79-547, 79-571 AND 79-574) IN THAT THE LIQUID WASTE FLOW RECORDER CHARTS INDICATE THAT THE VOLUMES DISCHARGED WERE BETWEEN 1800 AND 2500 GALLONS, WHILE THE MAXIMUM WHICH COULD BE RELEASED FOLLOWING THE PROCEDURES WOULD BE 1116 GALLONS SIMULTANEOUS RELEASE OF BOTH CONTAMINATED DRAIN TANKS. PREVENTIVE MAINTENANCE PROCEDURES HAVE NOT BEEN FOLLOWED IN THAT PREVENTIVE MAINTENANCE ON VARIOUS PIECES OF EQUIPMENT HAD NOT BEEN PERFORMED AT THE FREQUENCY ESTABLISHED BY THE PROCEDURES AND WAS OVERDUE AT THE TIME OF THE INVESTIGATION. EXAMPLES INCLUDE COMPONENT COOLING WATER PUMPS WHICH ARE REQUIRED BY PROCEDURE MMP-P-CC-002 TO RECEIVE PREVENTIVE MAINTENANCE ON AN ANNUAL BASIS. AS OF MAY 8, 1980 THREE OF THE FOUR PUMPS (B, C AND D) WERE OVERDUE BY AT LEAST SIX MONTHS. VARIOUS OPERATIONS FOR WHICH WRITTEN PROCEDURES HAVING INDIVIDUAL PROCEDURAL STEP SIGN-OFF IN ACCORDANCE WITH SECTION 5 OF THE VEPCO NUCLEAR POWER STATION QUALITY ASSURANCE MANUAL AND SURRY ADMINISTRATIVE PROCEDURE NO. 29, WERE PERFORMED WITHOUT THE APPROPRIATE PROCEDURE BEING SIGNED OFF AS THE INDIVIDUAL STEPS WERE ACCOMPLISHED. A SPECIFIC EXAMPLE INCLUDES THE RELEASES OF LIQUIDS FROM THE CONTAMINATED DRAIN TANKS.
(7990 INFRACTION)

PERSONNEL IN VISITOR STATUS FREQUENTLY FOUND WITHOUT ESCORT BY SECURITY OFFICERS WHILE WITHIN PA.
(7990 INFRACTION)

FAILURE TO PROVIDE EFFECTIVE COMPENSATORY MEASURES FOR EQUIPMENT OUTAGE.
(7990 INFRACTION)

FAILURE TO TEST EQUIPMENT IN ACCORDANCE WITH ESTABLISHED PROCEDURE.
(7990 INFRACTION)

AS REQUIRED BY SECTION 5.1.5 OF THE VEPCO NUCLEAR POWER STATION QUALITY ASSURANCE MANUAL THE SUPERVISOR OF ENGINEERING SERVICES OR HIS DESIGNEE IS REQUIRED TO REVIEW COMPLETED TEST RESULTS FOR COMPLETENESS, TO DETERMINE IF THE ESTABLISHED ACCEPTANCE CRITERIA HAD BEEN MET, AND TO RECOMMEND FINAL APPROVED BY THE STATION NUCLEAR SAFETY AND OPERATING COMMITTEE. CONTRARY TO THE ABOVE, THE TWO AUXILIARY STEAM GENERATOR FEED PUMPS, 2-FW-P-3A AND 2-FW-P-3B, FAILED TO MEET THE START TIME ACCEPTANCE CRITERIA OF STEPS 6.2.2.5 AND 6.4.3 OF ST-78 WITHOUT IDENTIFYING THE DISCREPANCY. IN ADDITION THE OUTSIDE RECIRCULATION SPRAY PUMP, 2-RS-P-2B FAILED TO START DUE TO AN IMPROPERLY RACKED-IN CIRCUIT BREAKER. RESOLUTION AS TO RETESTING OR REPERFORMING THE TEST WAS NOT IDENTIFIED.
(3024 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

SYSTEM MODIFICATIONS AND REANALYSIS FOR SEISMIC SUPPORTS ARE CONTINUING.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

UNIT IS CURRENTLY AT POWER AFTER CONDUCTING LOW POWER TESTING. THE UNIT HAS BEEN REFUELED.

LAST IE SITE INSPECTION DATE: AUGUST 1, 1980

INSPECTION REPORT NO: 50-281/80-32

SUMMARY

INSPECTION SUMMARY

INSPECTION JUNE 9 - JULY 3 (80-26): THIS ROUTINE INSPECTION BY THE RESIDENT INSPECTOR INVOLVED 45 INSPECTOR-HOURS ON SITE IN THE AREAS OF PLANT OPERATIONS AND OPERATING RECORDS, PLANT MODIFICATIONS, MAINTENANCE AND TESTING, IE BULLETINS, LICENSEE EVENT REPORTS, AND PLANT SECURITY. OF THE FIVE AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE FOUND IN FOUR AREAS; ONE APPARENT ITEM OF NONCOMPLIANCE WAS FOUND IN ONE AREA (INFRACTION - FAILURE TO FOLLOW PLANT MAINTENANCE AND MODIFICATION PROCEDURES).

INSPECTION JULY 9-10 (80-29): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 8 INSPECTOR-HOURS ON SITE IN THE AREAS OF THE IE BULLETIN 79-02, "PIPE SUPPORT BASE PLATE DESIGN USING CONCRETE EXPANSION ANCHOR BOLTS", IE BULLETIN 79-14", SEISMIC ANALYSIS FOR AS-BUILT SAFETY RELATED PIPING SYSTEMS" AND LICENSEE ACTIONS ON PREVIOUS INSPECTION FINDINGS. OF THE THREE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION JULY 24-25 (80-30): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 24 INSPECTOR-HOURS AT EBASCO SERVICES INCORPORATED, NEW YORK IN THE AREAS OF IE BULLETIN 79-14, "SEISMIC ANALYSIS FOR AS-BUILT SAFETY-RELATED PIPING SYSTEMS", IE BULLETIN 79-02, "PIPE SUPPORT BASE PLATE DESIGNS USING CONCRETE EXPANSION ANCHOR BOLTS", AND LICENSEE ACTIONS ON PREVIOUS INSPECTION FINDINGS. OF THE THE THREE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION JULY 21-25 (80-31): THIS SPECIAL, ANNOUNCED INSPECTION INVOLVED 30 INSPECTOR-HOURS ON SITE IN THE AREAS OF DOCUMENTATION REVIEW WITH RESPECT TO IE BULLETIN 79-01B RESPONSE. NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION JULY 29 - AUGUST 1 (80-32): THIS ROUTINE UNANNOUNCED INSPECTION INVOLVED 12 INSPECTOR-HOURS ON SITE IN THE AREAS OF LICENSEE EVENT REPORTS (LER'S) AND PRESERVICE INSPECTION RECORDS. NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

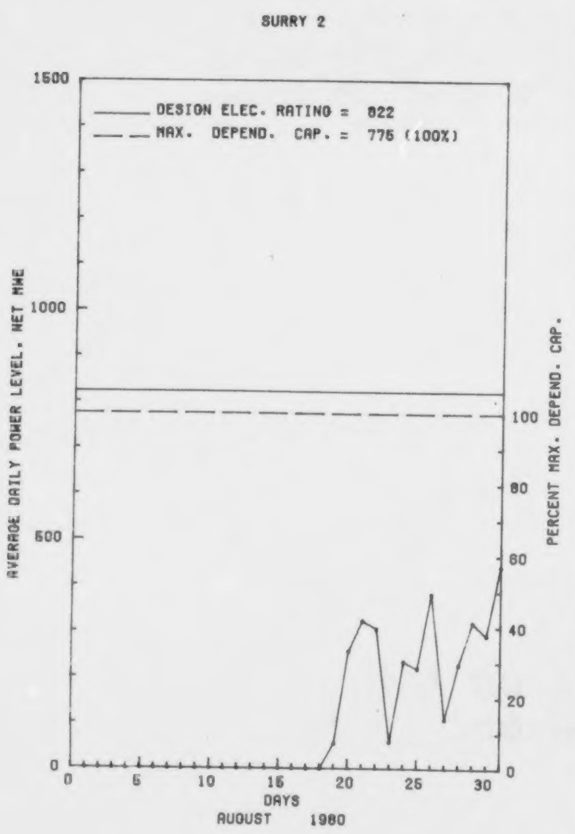
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-008/03L-0	06/23/80	07/11/80	AIR EJECTOR ALARM SETPOINT ABOVE T.S. LIMIT DUE TO INSTRUMENT DRIFT

REVIEWED BY OMPA LOEB: T.C. Cintula **DATE:** 10·6·80

REVIEWED BY NRR DL: Don Neighbors **DATE:** 10/7/80

SURRY 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-281
2. Utility Contact: SUE DUNN (804) 357-3184
3. Licensed Thermal Power (MWt): 2441
4. Nameplate Rating (Gross MWe): 942 x 0.9 = 848
5. Design Electrical Rating (Net MWe): 822
6. Maximum Dependable Capacity (Gross MWe): 811
7. Maximum Dependable Capacity (Net MWe): 775
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	5,855.0	64,319.0
12. Hours Reactor Was Critical	366.1	366.1	34,866.0
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	232.2	232.2	34,228.4
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	300,919	300,919	79,495,032
17. Gross Electrical Energy (MWH)	86,180	86,180	25,955,024
18. Net Electrical Energy (MWH)	77,485	77,485	24,614,090
19. Unit Service Factor	31.2	4.0	53.2
20. Unit Availability Factor	31.2	4.0	53.2
21. Unit Capacity Factor (MDC Net)	13.4	1.7	49.4
22. Unit Capacity Factor (DER Net)	12.7	1.6	46.6
23. Unit Forced Outage Rate	25.6	25.6	21.0

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-8	02/04/79	S	431.7	D	4				CONTINUATION OF SHUTDOWN FOR REFUELING AND STEAM GENERATOR REPLACEMENT, WHICH BEGAN ON 02/04/79. WORK STILL UNDERWAY ON PIPING RESTRAINT MODIFICATIONS REQUIRED PRIOR TO UNIT RECOVERY.
80-9	08/19/80	F	15.9	A	3				REACTOR TRIP CAUSED BY "B" S/G HIGH LEVEL AS A RESULT OF "B" MAIN FEED REGULATING VALVE FAILING OPEN. THE PROBLEM WAS FOUND TO BE A FAULTY CONTROLLER. THE CONTROLLER WAS REPLACED.
80-10	08/19/80	F	1.9	G	3				REACTOR TRIP CAUSED BY "A" S/G LO-LO LEVEL WHILE FEEDING S/G'S IN MANUAL. ANOTHER MORE EXPERIENCED OPERATOR WAS PLACED ON THE FEED CONTROL STATION.
80-11	08/22/80	F	23.6	A	3				REACTOR TRIP AND SAFETY INJECTION ON SPURIOUS STEAM HEADER TO STEAM LINE DELTA P SIGNAL CAUSED BY STEAM FROM LIFTING RELIEF VALVE IMPINGING ON STEAM HEADER PRESSURE TRANSMITTER. THE REASON FOR THE RV LIFTING WAS DETERMINED TO BE STUCK NRV'S IN STEAM DRAIN LINE. THE NRV'S WERE REPAIRED.
80-12	08/24/80	F	13.7	A	3				REACTOR TRIP CAUSED BY TURBINE TRIP AS A RESULT OF LOSS OF AUTO-STOP OIL PRESSURE THROUGH FAULTY RELIEF VALVE. THE FAULTY RELIEF VALVE WAS REPLACED.
80-13	08/26/80	F	12.1	H	3				FLOW SURGE CAUSED BY AIR TRAPPED IN CONDENSATE POLISHING SYSTEM AIR OPERATED BYPASS VALVE LINE RESULTED IN STEAM HEADER STEAMLINE DELTA P SAFETY INJECTION SIGNAL. THE AOV WILL BE OPERATED IN MANUAL TO REDUCE THE POSSIBILITY OF A FEED FLOW SURGE.
80-14	08/27/80	F	9.3	G	3				ELECTRICIANS WORKING IN INSTRUMENT RACKS INTERRUPTED POWER TO TRAIN "A" REACTOR TRIP RELAYS CAUSING REACTOR TRIP. THE ELECTRICIANS WERE INSTRUCTED TO INSURE PROPER PROCEDURES AND WORK PRACTICES ARE FOLLOWED WHEN WORKING IN INSTRUMENT RACKS.
80-15	08/30/80	F	3.6	G	3				REACTOR TRIP CAUSED BY "C" S/G HIGH LEVEL AS A RESULT OF "C" MAIN FEED REGULATING VALVE FAILING OPEN. THE INSTRUMENT TECHNICIANS WERE HOOKING UP A VISICORDER IN THE INSTRUMENT RACKS AND USED THE WRONG TYPE OF LEADS. THE INSTRUMENT TECHNICIANS WERE INSTRUCTED TO INSURE THE PROPER LEADS ARE USED IN THE FUTURE.

SUMMARY

SURRY 2 CAME ON-LINE ON THE 19TH OF AUGUST WITH NUMEROUS OUTAGES FOR THE MONTH LISTED IN DETAIL ABOVE.

LEGEND

[1] Type:
F–Forced
S–Scheduled

[2] Reason:
A–Equipment Failure
B–Maintenance or Test
C–Refueling
D–Regulatory Restriction
E–Operator Training & License Examination
F–Administrative
G–Operational Error
H–Other

[3] Method:
1–Manual
2–Manual Scram
3–Automatic Scram
4–Continuations
5–Load Reduction
9–Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

THREE MILE ISLAND 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Pennsylvania/Dauphin 10 miles SE of Harrisburg
2.	Electric Reliability Council	Mid Atlantic Area Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2535
5.	Design Electrical Rating (MWE Net)	819
6.	Date of Initial Criticality	June 5, 1974
7.	Date of Commercial Operation	September 2, 19[illegible]
8.	Condenser Cooling Method	Closed
9.	Condenser Cooling Water Source	Susquehanna River

Utility & Contractor Information

10.	Licensee	Metropolitan Edison Company
11.	Corporate Address	P. O. Box 542 Reading, Pennsylvania
12.	Corporate Contact	R. C. Arnold, Sr. Vice-President 260 Cherry Hill Rd. Parsippany, NJ 07054
13.	Architect/Engineer	Gilbert Associates, Inc.
14.	Nuclear Steam Supply System	B&W
15.	Constructor	UE&C

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Section Chief	A. N. Fasano
19.	NRC-TMI Support Staff	D. Haverkamp FTS: 590-3950 (717) 782-3950
20.	Licensing Project Manager	D. C. DiIanni / H. Silver (X27435)
21.	Docket Number	50-289
22.	License Number/Date of Issue	DPR-50, 4/19/74
23.	Public Document Room	Government Publications Section State Library of Pennsylvania Education Building Commonwealth and Walnut Street Harrisburg, Pennsylvania 17126

INSPECTION STATUS

ENFORCEMENT STATUS

FAILURE TO LIMIT ACCESS TO PROTECTED AREA AND VITAL AREA. (8003 INFRACTION)

FAILURE TO TEST PERSONNEL SEARCH EQUIPMENT. (8003 INFRACTION)

INSPECTION STATUS

. OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

COMPLETE REQUIRED MODIFICATIONS PRIOR TO RESTART.

PLANT STATUS:

The plant remains shutdown, by NRC Order, pending completion of modifications and other actions related to the TMI-2 accident.

LAST IE SITE INSPECTION DATE: 8/25-29/80

INSPECTION REPORT NO: 50-289/80-25

SUMMARY

INSPECTION SUMMARY

50-289/80-15 - JUL 14-18: ROUTINE, UNANNOUNCED INSPECTION BY A REGION-BASED INSPECTOR (37 HRS) OF THE PREOPERATIONAL AND STARTUP TEST RESTART PROGRAM INCLUDING TEST PROGRAM REQUIREMENTS, TEST PROCEDURES AND PROCEDURE REVIEWS, TEST SCHEDULING AND SEQUENCING, TEST PROGRAM ADMINISTRATION, RESTART MODIFICATIONS AND SCHEDULES; AND, TOUR OF THE FACILITY. NO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED.

50-289/80-17 - JUN 20 - JUL 10: ROUTINE, UNANNOUNCED INSPECTION BY FIVE REGION-BASED INSPECTORS (107 HRS) OF WORK ACTIVITIES AND RECORDS ASSOCIATED WITH THE RESTART PROGRAM AS WELL AS CERTAIN PREVIOUSLY UNRESOLVED ITEMS INCLUDING: THE TENDON SURVEILLANCE PROGRAM, THE BUTT-SPLICE PROGRAM (TASK RM-1), THE FIRE PROTECTION PROGRAM (TASK NM-40), THE HIGH-PRESSURE INJECTION CROSS-CONNECT MODIFICATION (TASK RM-14), QUALIFICATIONS OF QC INSPECTORS, AND QA SITE AUDITS. NO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

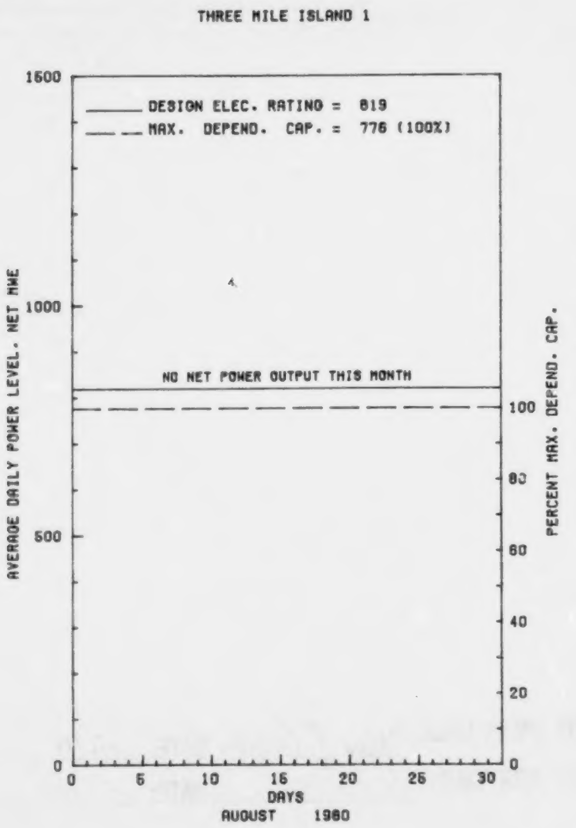
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-008/ 99X	04/30/80	08/04/80	DETERIORATION OF THE 'A' EMERGENCY DIESEL GENERATOR BEARING INSULATION WHICH COULD LEAD TO POSSIBLE GENERATOR BEARING FAILURE.
80-009/ 03L	06/07/80	06/25/80	REACTOR BUILDING TENDON SURVEILLANCE DATE WAS EXCEEDED PRIOR TO COMPLETING THE SURVEILLANCE.
80-011/ 03L	05/14/80	07/11/80	FAILURE TO PERFORM THE REACTOR BUILDING SPRAY SYSTEM COMPRESSED AIR TEST WITHIN THE ALLOWABLE TIME LIMIT.
80-012/ 01T	06/27/80	07/11/80	HIGHER THAN PREVIOUSLY ANALYZED POST ACCIDENT PRESSURE AND TEMPERATURE IN THE INTERMEDIATE BUILDING FOLLOWING A POSTULATED MAIN STEAM LINE BREAK.
80-015/ 04P	07/21/80	07/22/80	RIVER WATER DISCHARGE TEMPERATURE WAS MORE THAN 3 DEGREES F BELOW THE INLET RIVER WATER TEMPERATURE FOR APPROX. SIX HOURS WHICH WAS A VIOLATION OF ETS 2.1.A(1).

REVIEWED BY OMPA LOEB: John L Crooks **DATE:** 10/6/80

REVIEWED BY NRR DL: D CD. **DATE:** 10/8/80

THREE MILE ISLAND 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-289
2. Utility Contact: D. G. Mitcneil (215) 921-6579
3. Licensed Thermal Power (MWt): 2535
4. Nameplate Rating (Gross MWe): 968 x 0.9 = 871
5. Design Electrical Rating (Net MWe): 819
6. Maximum Dependable Capacity (Gross MWe): 840
7. Maximum Dependable Capacity (Net MWe): 776
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	5,855.0	52,584.0
12. Hours Reactor Was Critical	.0	.0	31,731.8
13. Reactor Reserve Shutdown Hours	.0	.0	838.5
14. Hours Generator On-Line	.0	.0	31,180.9
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	0	76,531,071
17. Gross Electrical Energy (MWH)	0	0	25,484,330
18. Net Electrical Energy (MWH)	0	0	23,846,353
19. Unit Service Factor	.0	.0	59.3
20. Unit Availability Factor	.0	.0	59.3
21. Unit Capacity Factor (MDC Net)	.0	.0	58.4
22. Unit Capacity Factor (DER Net)	.0	.0	55.4
23. Unit Forced Outage Rate	100.0	100.0	31.0

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	02/17/79	F	744.0	D	4				REGULATORY RESTRAINT ORDER.

SUMMARY

THREE MILE ISLAND 1 REMAINS SHUTDOWN FOLLOWING THE ACCIDENT AT THREE MILE ISLAND 2.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

THREE MILE ISLAND 2

NRC POWER RESTRICTIONS: SEE "LICENSE STATUS"

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Pennsylvania/Dauphin 10 miles SE of Harrisburg
2.	Electric Reliability Council	Mid-Atlantic Area Council
3.	Type of Reactor	PWR
4.	Licenses Power Level (MWT)	2772
5.	Design Electrical Rating (MWE Net)	906
6.	Date of Initial Criticality	3/28/78
7.	Date of Commercial Operation	12/30/78
8.	Condenser Cooling Method	CLOSED
9.	Condenser Cooling Water Source	Susquehanna River

Utility & Contractor Information

10.	Licensee	Metropolitan Edison Company
11.	Corporate Address	P. O. Box 542 Reading, PA 19603
12.	Corporate Contact	R. C. Arnold, Sr. Vice-President 100 Interpace Parkway Parsippany, NJ 07054
13.	Architect/Engineer	Burns & Roe
14.	Nuclear Steam Supply System	B&W
15.	Constructor	UE&C

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Section Chief	FTS: 590-3950 A. N. Fasano (717) 782-3950
19.	IE Resident Inspector	FTS: 590-3950 R. Conte (717) 782-3950
20.	TMI Program Director	B. Snyder
21.	Docket Number	50-320
22.	License Number/Date of Issue	DPR-73/2; 2/8/78
23.	Public Document Room	Government Publications Section State Library of Pennsylvania Education Building Commonwealth & Walnut Streets Harrisburg, Pennsylvania 17126

INSPECTION STATUS

ENFORCEMENT STATUS

FAILURE TO IMPLEMENT PROCEDURES FOR CONTROLLING ACCESS TO PROTECTED AREA AND VITAL AREA.
(8003 INFRACTION)

FAILURE TO IMPLEMENT BADGE CONTROL PROCEDURE.
(8003 INFRACTION)

CONTRARY TO 10 CFR 50.72 BETWEEN MARCH 20, 1980, AND MAY 29, 1980, REQUIRED REPORTS TO THE NRC OPERATIONS CENTER WERE NOT PROPERLY MADE: MARCH 20, 1980, HIGH AIRBORNE IN AUX BUILDING 10 CFR 50.72(1) REPORT NOT MADE; MAY 20, 1980, CONTAINMENT INTEGRITY NOT MAINTAINED REPORT MADE IN EXCESS OF 1 HOUR (10 CFR 50.72(C)); MAY 29, 1980 RCS PRESSURE ISOLATION INADVERTENT, NO 10 CFR 50.72(3) REPORT MADE.
(8009 DEFICIENCY)

CONTRARY TO TS 3.3.3.6 (TABLE 3.3-10) (ORDERS OF 7/20/79 AND 2/11/80) ON MAY 29, 1980, THE ONLY RCS PRESSURE SOURCE WAS NOT OPERABLE BECAUSE OF THE INADVERTENT CLOSURE OF A RECOVERY SYSTEM INTERFACE VALVE (SNS-V2C) WHICH ISOLATED TWO PRESSURE READING INSTRUMENTS FROM THE RCS. THIS RESULTED BECAUSE A PROCEDURE (SPECIAL OPERATING PROCEDURE R-2-8C-18, DATED APRIL 3, 1980, TEMPORARY NUCLEAR SAMPLE SYSTEM FLOWPATH VERIFICATION) UTILIZED VALVE LINEUP REQUIRING SNS-V2C TO BE SHUT.
(8009 INFRACTION)

CONTRARY TO TS 3.6.1.1 (ORDERS OF 7/20/79 AND 2/11/80) ON MAY 20, 1980, BETWEEN 9:00 P.M. TO 9:30 P.M. FOR APPROXIMATELY 5-10 MINUTES CONTAINMENT INTEGRITY WAS NOT MAINTAINED AT REACTOR BUILDING PERSONNEL AIRLOCK NO. 2 WITH THE OUTER DOOR MANUAL PURGE VALVE (DESIGNATED "E" VALVE) AND THE INNER DOOR MANUAL EQUALIZATION VALVE OPEN SIMULTANEOUSLY. THIS RESULTED BECAUSE A PROCEDURE (OPERATING PROCEDURE 2104-4.55, REVISION 1, MAY 16, 1980 REACTOR BUILDING ENTRY AND PRE-DECON) APPROVED PURSUANT TO SPECIFICATION 6.8.2, REQUIRED THE OPENING OF THE OUTER AIRLOCK DOOR IN ACCORDANCE WITH ANOTHER REFERENCED PROCEDURE BUT NEITHER PROCEDURE ADDRESSED THE SHUTTING OF THE "E" VALVE ON THE OUTER DOOR IN THE PROPER SEQUENCE.
(8009 INFRACTION)

CONTRARY TO TS 6.8.1, RG 1.33, PROCEDURE 1410-7-26 PARA. 6.1.1 AND 6.1.4 ON MARCH 29, 1980, CONTRACT PERSONNEL CONDUCTED OPEN FLAME CUTTING IN THE FUEL HANDLING BUILDING WITHOUT IMPLEMENTING THE FIRE PROTECTION PERMIT SYSTEM. AS A RESULT OF THIS, THE CONTROL ROOM WAS NOT AWARE THAT SUCH OPERATIONS WERE OCCURRING: SPARKS WERE SPANNING AN AREA 20-30 FEET FROM THE SOURCE CUTTING WITH FLAMMABLE BOXES WITHIN 5 FEET AND WITH THE CUTTING DONE ON NONTREATED WOOD; NO FIRE EXTINGUISHERS WERE DESIGNATED FOR USE IN THIS PARTICULAR OPERATION, AND THERE WAS NO DOCUMENTATION OF FOLLOWUP BY THE DESIGNATED FIREWATCH.
(8009 INFRACTION)

10 CFR 71.5(A) REQUIRES THAT NRC LICENSEES COMPLY WITH THE APPLICABLE PACKAGING AND TRANSPORTATION REQUIREMENTS OF THE DEPARTMENT OF TRANSPORTATION (DOT) IN 49 CFR PARTS 170-189. A. 49 CFR 173.395(A)(1) REQUIRES THAT MATERIALS BE PACKAGED IN ACCORDANCE WITH DOT SPECIFICATION 7A, TYPE A PACKAGING, AND THAT EACH SHIPPER OF A SPECIFICATION 7A PACKAGE MAINTAIN ON FILE A COMPLETE CERTIFICATION AND SUPPORTING SAFETY ANALYSIS DEMONSTRATING COMPLIANCE WITH THE SPECIFICATION. CONTRARY TO THE ABOVE, ON FEBRUARY 6 AND MARCH 6, 1980, LIQUID RADIOACTIVE MATERIAL WAS DELIVERED IN TYPE A QUANTITIES TO A CARRIER FOR TRANSPORT IN CONTAINERS WHICH WERE NOT AUTHORIZED BY THE SUPPORTING SAFETY ANALYSIS FOR THE SHIPMENT OF LIQUIDS. THE SAFETY ANALYSIS ONLY AUTHORIZED THE PACKAGING OF SOLID RADIOACTIVE MATERIAL.
(8010 VIOLATION)

10 CFR 71.5(A) REQUIRES THAT NRC LICENSEES COMPLY WITH THE APPLICABLE PACKAGING AND TRANSPORTATION REQUIREMENTS OF THE DEPARTMENT OF TRANSPORTATION (DOT) IN 49 CFR PARTS 170-189. B. 49 CFR 173.393(G) REQUIRES LIQUID RADIOACTIVE MATERIAL IN TYPE A QUANTITIES BE PACKAGED IN OR WITHIN A LEAK-RESISTANT AND CORROSION RESISTANT INNER CONTAINMENT VESSEL. CONTRARY TO THE ABOVE, ON FEBRUARY 6 AND MARCH 6, 1980, THE INNER CONTAINMENT VESSELS OF THE PACKAGES LEAKED RADIOACTIVE MATERIAL DURING TRANSPORT DEMONSTRATING THAT THE INNER CONTAINERS WERE NOT LEAK-RESISTANT. IN THE FEBRUARY 6 SHIPMENT, THE VALVE HANDLES WERE NOT REMOVED AND WERE LEFT UNPROTECTED ON THE SAMPLE BOMB, THE INNER CONTAINMENT VESSEL, RESULTING IN THE LEAKAGE. IN THE MARCH 6 SHIPMENT, OF THE TEN POLYETHYLENE BOTTLES SHIPPED AS INNER CONTAINMENT VESSELS, ONE WAS CRUSHED AND THREE OTHERS LEAKED.
(8010 VIOLATION)

OTHER ITEMS OF INTEREST

SYSTEM AND COMPONENT PROBLEMS:

Cleanup operations from 3/28/79 accident.

PLANT STATUS:

Decay heat is being removed by cyclic natural circulation through the "A" Once Through Steam Generator and to the reactor building ambient from both RCS loops.

LAST IE SITE INSPECTION DATE: 8/31/80

INSPECTION REPORT NO: 50-320/80-12

SUMMARY

INSPECTION SUMMARY

50-320/80-09 - APR 26 - MAY 31: ROUTINE INSPECTIONS BY NRC TMI PROGRAM OFFICE STAFF OF LICENSEE ACTION ON PREVIOUS INSPECTION FINDINGS; PROCEDURE IMPLEMENTATION; SELECTED NEW AND REVISED PROCEDURES SUBMITTED FOR APPROVAL; IMPLEMENTATION OF NOTIFICATION OF SIGNIFICANT EVENT REQUIREMENTS; AND, HEALTH PHYSICS AND ENVIRONMENTAL AREAS. THE INSPECTION INCLUDED DAILY (MONDAY-FRIDAY) ONSITE STAFF COVERAGE WITH SELECTED BACKSHIFT COVERAGE. FOUR ITEMS OF NONCOMPLIANCE WERE IDENTIFIED: FAILURE TO IMPLEMENT FIRE PROTECTION MEASURES DURING OPEN FLAME CUTTING IN FUEL HANDLING BUILDING; FAILURE TO MAINTAIN CONTAINMENT INTEGRITY; FAILURE TO MAINTAIN REACTOR COOLANT SYSTEM PRESSURE INDICATION OPERABLE; FAILURE TO PROPERLY REPORT SIGNIFICANT EVENTS.

50-320/80-10 - MAR 17 - APR 11: SPECIAL, UNANNOUNCED INSPECTION BY ONE RESIDENT INSPECTOR OF SHIPMENTS OF RADIOACTIVE LIQUID SAMPLES TO BABCOCK AND WILCOX AND OAK RIDGE NATIONAL LABORATORY INCLUDING REVIEW OF DISCREPANT SHIPMENTS AND PACKAGING SAFETY ANALYSIS. TWO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED: FAILURE TO USE LEAK RESISTANT INNER CONTAINMENT VESSEL FOR RADIOACTIVE LIQUID SHIPMENTS; FAILURE TO COMPLY WITH AUTHORIZED PACKAGING FOR A TYPE A SHIPMENT.

50-320/80-14 - JUN 9-13 (15000046/80-72): INSPECTION OF TRAILER 440571, LOADED WITH RADIOACTIVE WASTE PACKAGES. ONE PACKAGE WAS OPENED. A REPRESENTATIVE FROM THE STATE OF WASHINGTON WHO WAS ON SITE DURING THE ENTIRE INSPECTION OPENED AND EXAMINED THE PACKAGE. THE PACKAGE CONFORMED TO THE DESCRIPTION IN THE RSR AND CONTAINED NO FREE STANDING LIQUID. ONE ITEM OF NONCOMPLIANCE WITH DOT/NRC REGULATIONS WAS IDENTIFIED: FAILURE

LICENSE STATUS

By Order for Modification of License dated July 20, 1979, the licensee's authority to operate the facility was suspended and the licensee's authority was limited to maintenance of the facility in the present shutdown cooling mode (44 F.R. 45271).

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-016/ 03L	05/01/80	05/30/80	LOW OUT OF SPEC FUEL OIL LEVEL FOR GRAY BOP DIESEL DUE TO OPERATOR ERRONEOUS LEVEL READING AND THEREFORE NO OIL ADDITION WAS MADE.
80-018/ 03L	05/09/80	06/10/80	NO VOLTAGE OUTPUT ON WHITE BOP DIESEL DUE DISCONNECTED POTENTIAL TRANSFORMER FROM SHOCK ON PREVIOUS TRIP.

REVIEWED BY OMPA LOEB: John F Crooks **DATE:** 10/6/80

REVIEWED BY NRR TMIPO D. J. Brinkman **DATE:** 10/7/80

THREE MILE ISLAND 2

AVERAGE DAILY POWER LEVEL (MWe)

OPERATING STATUS

1. Docket Number: 50-320
2. Utility Data Prepared By: R. A. Lengel (215) 921-6581
3. Licensed Thermal Power (MWt): 2772
4. Nameplate Rating (Gross MWe): 1068 X 0.9 = 961
5. Design Electrical Rating (Net MWe): 906
6. Maximum Dependable Capacity (Gross MWe): 0
7. Maximum Dependable Capacity (Net MWe): 0
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons

None

9. Power Level To Which Restricted, If Any (Net MWe) None
10. Reasons For Restrictions, If Any:

This Month Yr

11. Hours In Reporting Period
12. Number Of Hours Reactor Was Critical
13. Reactor Reserve Shutdown Hours
14. Hours Generator On-Line
15. Unit Reserve Sh
16. Gro

(MWH)

19

20. U Availability Factor
21. Unit Capacity Factor (Using MDC Net) 1/
22. Unit Capacity Factor (Using DER Net) 1/
23. Unit Forced Outage Rate
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

None

25. If Shut Down At End Of Report Period, Estimated Date of Startup: N/A

NO DECISION ON FUTURE OPERATION
SHUTDOWN INDEFINITELY

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
SEE SUMMARY BELOW.									

SUMMARY

THE UNIT HAS REMAINED SHUTDOWN SINCE THE 03/28/79 ACCIDENT, WHICH CAUSED EXTENSIVE CORE DAMAGE.

AUTHORITY TO OPERATE FACILITY SUSPENDED BY NRC ORDER DATED JULY 20, 1979, WHICH REQUIRES LICENSEE TO MAINTAIN FACILITY IN A SHUTDOWN CONDITION.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

TROJAN

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Oregon/Columbia 42 miles N of Portland
2.	Electric Reliability Council	Western Systems Coordinating Council
3.	Type of Reactor	PWR
4.	Licenses Power Level (MWT)	3411
5.	Design Electrical Rating (MWE Net)	1130
6.	Date of Initial Criticality	December 15, 1975
7.	Date of Commercial Operation	May 20, 1976
8.	Condenser Cooling Method	Natural draft cooling tower
9.	Condenser Cooling Water Source	Columbia River

Utility & Contractor Information

10.	Licensee	Portland General Electric Co.
11.	Corporate Address	121 S. W. Salmon St. Portland, Oregon 97204
12.	Corporate Contact	C. Goodwin, Jr., Asst. V.P.
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Multiple

Regulatory Information

16.	IE Region responsible	V
17.	IE Branch Chief	J. L. Crews
18.	IE Section Chief	D. M. Sternberg
19.	IE Resident Inspector	M. H. Malmros (503) 556-3751
20.	Licensing Project Manager	C. M. Trammell
21.	Docket Number	50-344
22.	License Number/Date of Issue	NPF-1, 11/21/75
23.	Public Document Room	Columbia County Courthouse Law Library Circuit Court Room St. Helens, Oregon 97501

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO 10 CFR 71.5(A) AND 49 CFR 173.392(C)(8), TWO CONTAINERS NUMBERED 363 AND 409 IN YOUR SHIPMENT DATED JUNE 18, 1980 DID NOT HAVE THE MARKING "RADIOACTIVE - LSA" ON THE OUTSIDE OF THE PACKAGE. (8017 INFRACTION)

CONTRARY TO 10 CFR 71.5(A) AND 49 CFR 173.392(C)(1), ONE CONTAINER NUMBERED 365 IN YOUR SHIPMENT DATED JUNE 18, 1980 WAS OBSERVED TO DRIP LIQUID DURING OFF-LOADING AT THE NUCLEAR ENGINEERING COMPANY, RICHLAND, WASHINGTON DISPOSAL SITE. WIPES TAKEN OF THE LEAKED LIQUID AND CONTAINER INDICATED NO REMOVABLE RADIOACTIVE CONTAMINATION. (8017 DEFICIENCY)

INSPECTION STATUS

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

PURSUANT TO THE ASLB DECISION OF JULY 11, 1980, A LICENSE AMENDMENT WAS ISSUED ON JULY 25, 1980 AUTHORIZING THE LICENSEE TO MODIFY THE CONTROL BUILDING TO RESTORE THE INITIAL DESIGN MARGINS.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS

ROUTINE OPERATIONS

LAST IE SITE INSPECTION DATE: 08/01-31/80+

INSPECTION REPORT NO: 50-344/80-21+

SUMMARY

INSPECTION SUMMARY:

INSPECTION ON JULY 14-17, 21, 22 & AUGUST 11, 1980 (REPORT NO. 50-344/80-15)
AREAS INSPECTED: INCLUDED A REVIEW OF SECURITY PLAN; SECURITY ORGANIZATION; VITAL AREA PHYSICAL BARRIERS; LIGHTING; ACCESS CONTROL (SEARCH); ACCESS CONTROL (ESCORTING); ASSESSMENT AIDS; COMPENSATORY MEASURES; RESPONSE; RECORDS AND REPORTS; FOLLOWUP ON PREVIOUS ITEMS OF NONCOMPLIANCE; AND FOLLOWUP ON UNRESOLVED ITEMS. THE INSPECTION INVOLVED 77 INSPECTOR HOURS ONSITE BY TWO NRC INSPECTORS.

RESULTS: THE LICENSEE WAS FOUND TO BE IN COMPLIANCE WITH NRC REQUIREMENTS WITHIN THE 10 PROGRAM AREAS EXAMINED DURING THIS INSPECTION, EXCEPT FOR THE FOLLOWING ITEM: COMPENSATORY MEASURES - FAILURE TO PROVIDE COMPLETE CLOSED CIRCUIT TV OR OTHER COMPENSATORY MEASURE FOR LACK OF GUARD TOWERS AND THEIR ASSOCIATED FUNCTIONS AS DESCRIBED IN THE APPROVED SECURITY PLAN.

INSPECTION ON JULY 7-18, 1980 (REPORT NO. 50-344/80-16)
REPORT BEING PREPARED; TO BE REPORTED NEXT MONTH.

INSPECTION ON JULY 22-25, 1980 (REPORT NO. 50-344/80-18)
AREAS INSPECTED: SPECIAL, ANNOUNCED PERFORMANCE APPRAISAL INSPECTION OF THE LICENSEE'S PROGRAM AND DOCUMENTATION ASSOCIATED WITH THE CONTROL OF CENTRIFUGAL CHARGING/SAFETY INJECTION PUMPS MANUFACTURED BY PACIFIC PUMP DIVISION OF DRESSER INDUSTRIES. THE INSPECTION INVOLVED 30 INSPECTOR HOURS ONSITE BY ONE PERFORMANCE APPRAISAL BRANCH INSPECTOR.

RESULTS: NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION ON JULY 21-25, 1980 (REPORT NO. 50-344/80-19)
AREAS INSPECTED: ROUTINE, UNANNOUNCED INSPECTION OF PROCUREMENT; RECEIPT, STORAGE AND HANDLING; SURVEILLANCE OF PIPE SUPPORTS AND RESTRAINTS; INSTRUMENT CALIBRATIONS; RESPONSE TO IE BULLETINS AND CIRCULARS; AND FOLLOWUP ON PREVIOUS INSPECTION ITEMS. ALSO INCLUDED INDEPENDENT INSPECTION EFFORT. THE INSPECTION INVOLVED 62 INSPECTOR HOURS ONSITE BY TWO NRC INSPECTORS.

RESULTS: NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION ON JULY 1-31, 1980 (REPORT NO. 50-344/80-20)
AREAS INSPECTED: ROUTINE INSPECTIONS OF PLANT OPERATIONS, SURVEILLANCE TESTING, PHYSICAL SECURITY, REFUELING ACTIVITIES, STARTUP TESTING, TRAINING, AND FOLLOWUP ON LICENSEE EVENT REPORTS. THE INSPECTION INVOLVED 151 INSPECTOR HOURS ONSITE BY THE NRC RESIDENT INSPECTORS.

RESULTS: ONE ITEM OF NONCOMPLIANCE WAS IDENTIFIED (INFRACTION - FAILURE TO FOLLOW A PROCEDURE RELATED TO DECONTAMINATION OF STEAM GENERATOR CHANNEL HEADS.

INSPECTION ON AUGUST 1-31, 1980 (REPORT NO. 50-344/80-21)
REPORT BEING PREPARED; TO BE REPORTED NEXT MONTH.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

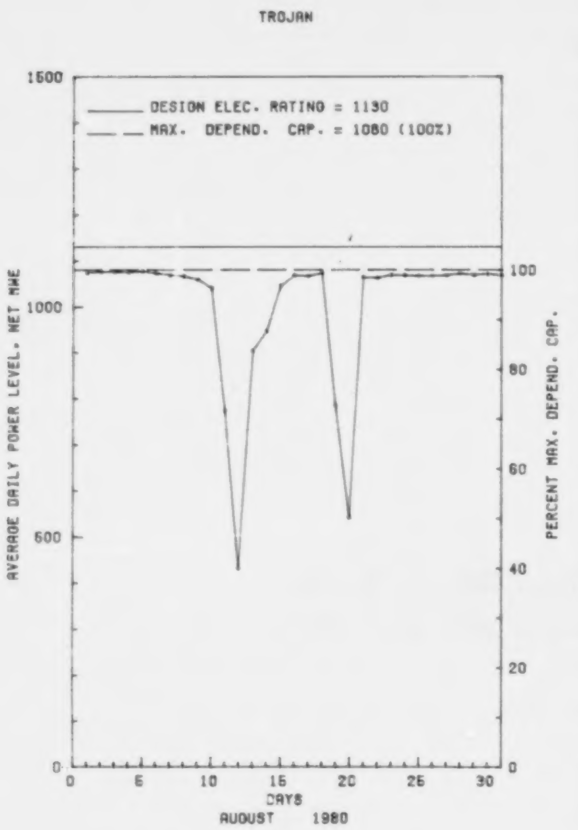
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-12/ 03L-0	07-01-80	07-16-80	MISSING RESTRAINTS ON STEAM DRIVEN AUXILIARY FEED PUMP STEAM DRAIN LINE.
80-13/ 03L-0	07-08-80	08-06-80	CUMULATIVE CONTAINMENT ISOLATION VALVE LEAKAGE IN EXCESS OF LIMITS.
80-14/ 03L-0	07-09-80	08-07-80	SEISMIC RESTRAINT ON CONTROL ROOM COOLING COIL INSTALLED IMPROPERLY.
80-15/ 03L-0	07-20-80	08-18-80	STEAM DRIVEN AUXILIARY FEED PUMP START FAILURE (THROTTLE VALVE TORQUE SW).
80-16/ 03L-0	[illegible]-19-80	08-18-80	BORIC ACID CONCENTRATION AND LEVEL OUTSIDE TECHNICAL SPECIFICATION LIMITS.

REVIEWED BY OMPA LOEB: T.C. Cintula **DATE:** 10.6.80

REVIEWED BY NRR DL: P C Wagner **DATE:** 10/8/80

TROJAN

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-344
2. Utility Contact: G. G. Bair (503) 556-3713 x234
3. Licensed Thermal Power (MWt): 3411
4. Nameplate Rating (Gross MWe): 1280 x 0.95 = 1216
5. Design Electrical Rating (Net MWe): 1130
6. Maximum Dependable Capacity (Gross MWe): 1122
7. Maximum Dependable Capacity (Net MWe): 1080
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	5,855.0	35,063.0
12. Hours Reactor Was Critical	744.0	3,537.4	19,835.4
13. Reactor Reserve Shutdown Hours	.0	.0	2,171.8
14. Hours Generator On-Line	744.0	3,443.3	19,071.1
15. Unit Reserve Shutdown Hours	.0	.0	1,508.7
16. Gross Thermal Energy (MWH)	2,399,173	9,980,127	58,875,737
17. Gross Electrical Energy (MWH)	781,140	3,214,315	19,199,730
18. Net Electrical Energy (MWH)	745,556	3,043,037	18,076,356
19. Unit Service Factor	100.0	58.8	54.4
20. Unit Availability Factor	100.0	58.8	58.7
21. Unit Capacity Factor (MDC Net)	92.8	48.1	47.7
22. Unit Capacity Factor (DER Net)	88.7	46.0	45.6
23. Unit Forced Outage Rate	.0	16.3	25.4

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): DESIGN MODIFICATIONS FROM TMI, OCTOBER 1980, 30 DAYS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-07	08/11/80	F	.0	A	5				CONDENSER TUBE LEAKS REQUIRED LOAD REDUCTION FROM 100% POWER TO 50% TO PERMIT ENTRY INTO THE CONDENSER AND PLUGGING OF SIX TUBES.
80-08	08/19/80	F	.0	G	5				REDUCE LOAD FROM 100% POWER TO 40% TO PLUG ADDITIONAL 23 CONDENSER TUBE LEAKS. THE REASON FOR CONDENSER LEAK INDICATIONS TURNED OUT TO BE A TUBE LEAK IN THE PROCESS STEAM VENT CONDENSER IN THE PROCESS STEAM SYSTEM.

SUMMARY

TROJAN OPERATED AT NEAR FULL POWER, WITH 2 REDUCTIONS TO PLUG CONDENSER TUBE LEAKS.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

TURKEY POINT 3

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Florida/Dade 25 miles S of Miami
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2200
5.	Design Electrical Rating (MWE Net)	693
6.	Date of Initial Criticality	October 20, 1972
7.	Date of Commercial Operation	December 14, 1972
8.	Condenser Cooling Method	Recirculating Canal System
9.	Condenser Cooling Water Source	Canal System

Utility & Contractor Information

10.	Licensee	Florida Power and Light Company
11.	Corporate Address	9250 West Flagler Street P. O. Box 013100 Miami, Florida 33101
12.	Corporate Contact	R. E. Uhrig, V.P. Nuclear Affairs
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. Lewis
18.	IE Section Chief	R. D. Martin
19.	IE Resident Inspector	R. J. Vogt-Lowell (305) 245-7669
20.	Licensing Project Manager	M. Grotenhuis (301) 492-7128
21.	Docket Number	50-250
22.	License Number/Date of Issue	DPR-31, 7/19/72
23.	Public Document Room	Environmental and Urban Affairs Library Florida International University Miami, Florida 33199

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

NORMAL OPERATION AT 100% POWER.

LAST IE SITE INSPECTION DATE: SEPTEMBER 9-12, 1980

INSPECTION REPORT NO: 50-250/80-27

SUMMARY

INSPECTION SUMMARY

INSPECTION MAY 5-16 (80-17): THIS SPECIAL, ANNOUNCED INSPECTION INVOLVED 235 INSPECTOR-HOURS ON SITE IN THE AREA OF HEALTH PHYSICS INCLUDING ORGANIZATION, QUALIFICATIONS, TRAINING PROCEDURES, ALARA PROGRAMS, EXTERNAL EXPOSURE CONTROL, PERSONNEL DOSIMETER PROGRAM, INTERNAL DOSIMETRY, RESPIRATORY PROTECTION, MEDICAL EMERGENCIES, INSTRUMENTATION, SURVEILLANCE AND ACCESS CONTROL, RADWASTE CONTROL, FACILITIES AND EQUIPMENT, RE-ENTRY AND IN-PLANT SYSTEMS. OF THE 16 AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN 13 AREAS; THREE ITEMS OF NONCOMPLIANCE WERE FOUND IN THREE AREAS.

INSPECTION JUNE 1-30 (80-26): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 64 INSPECTOR-HOURS ON SITE IN THE AREAS OF FOLLOWUP ON IMPLEMENTATION OF CATEGORY "A" SHORT TERM LESSONS LEARNED, PLANT OPERATIONS, SURVEILLANCE TEST OBSERVATION, AND PLANT TOURS. OF THE FOUR AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN THREE AREAS; ONE APPARENT ITEM OF NONCOMPLIANCE WAS FOUND IN ONE AREA (DEFICIENCY - IMPROPER STORAGE OF RADIOACTIVE MATERIAL).

REPORTS RECEIVED FROM LICENSEE

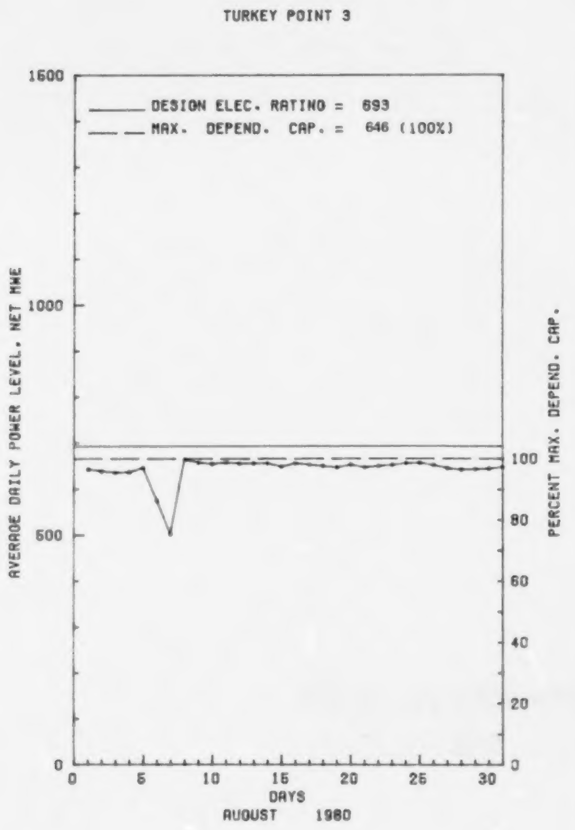
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-008/01X	05/05/80	08/11/80	INADEQUATE SUPPORT OF 3A AND 3B RTD BYPASS LOOPS
80-010/03L	05/14/80	06/13/80	TWO PROCESS RADIATION MONITORS OUT OF SERVICE DUE TO FAILURE OF AIR SAMPLING BLOWER
80-013/03L	06/24/80	07/08/80	INTERMITTENT CONTACT OPERATION OF WESTINGHOUSE W2 SWITCH IN NEUTRAL POSITION
80-014/01T	06/26/80	07/10/80	INSPECTION REVEALED A DEFICIENCY OF THE AFP SUCTION PIPING AND THE MS AFP TURBINE PIPING SUPPORTS

REVIEWED BY OMPA LOEB: T.C. CINTULA DATE: 10·6·80

REVIEWED BY NRR DL: M Grotenhuis DATE: 10-7-80

TURKEY POINT 3

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-250
2. Utility Contact: V. T. Chilson (305) 552-3824
3. Licensed Thermal Power (MWt): 2200
4. Nameplate Rating (Gross MWe): 894 x 0.85 = 760
5. Design Electrical Rating (Net MWe): 693
6. Maximum Dependable Capacity (Gross MWe): 680
7. Maximum Dependable Capacity (Net MWe): 646

ITEM 21 CALCULATED BY USING A WEIGHTED AVERAGE

8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: ITEMS 6 & 7 RERATED TO MOST RESTRICTIVE SEASON
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	5,855.0	67,856.6
12. Hours Reactor Was Critical	744.0	4,979.8	52,249.2
13. Reactor Reserve Shutdown Hours	.0	.0	3.3
14. Hours Generator On-Line	737.9	4,809.6	50,499.8
15. Unit Reserve Shutdown Hours	.0	.0	121.8
16. Gross Thermal Energy (MWH)	1,613,090	10,410,368	101,879,002
17 Gross Electrical Energy (MWH)	503,570	3,275,415	32,356,415
18. Net Electrical Energy (MWH)	478,174	3,104,278	30,626,372
19. Unit Service Factor	99.2	82.1	74.4
20. Unit Availability Factor	99.2	82.1	74.6
21. Unit Capacity Factor (MDC Net)	99.5	82.1	70.2
22. Unit Capacity Factor (DER Net)	92.7	76.5	65.1
23. Unit Forced Outage Rate	.8	1.3	2.6

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): STEAM GENERATOR TUBE INSPECTION - OCT. 5 - NOV. 1, 1980.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
16	08/06/80	F	6.1	B	1		CB	MOTORX	UNIT NO. 3 WAS REMOVED FROM SERVICE DUE TO LOW OIL LEVEL IN REACTOR COOLANT PUMP MOTOR NO. 3C. CORRECTIVE ACTIONS INCLUDED ADDING OIL.

SUMMARY

TURKEY POINT 3 OPERATED AT ABOVE 90% POWER FOR THE MONTH, WITH ONE OUTAGE ON THE 6TH TO SERVICE A REACTOR COOLANT PUMP MOTOR.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

TURKEY POINT 4

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Florida/Dade 25 miles S of Miami
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2200
5.	Design Electrical Rating (MWE Net)	693
6.	Date of Initial Criticality	June 11, 1973
7.	Date of Commercial Operation	September 7, 1973
8.	Condenser Cooling Method	Recirculating Canal System
9.	Condenser Cooling Water Source	Canal System

Utility & Contractor Information

10.	Licensee	Florida Power and Light Company
11.	Corporate Address	9250 West Flagler Street P. O. Box 013100 Miami, Florida 33101
12.	Corporate Contact	R. E. Uhrig, V. P. Nuclear Affairs
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. Lewis
18.	IE Section Chief	R. D. Martin
19.	IE Resident Inspector	R. J. Vogt-Lowell (305) 245-7669
20.	Licensing Project Manager	M. Grotenhuis (301) 492-7128
21.	Docket Number	50-251
22.	License Number/Date of Issue	DPR-41, 4/10/73
23.	Public Document Room	Environmental and Urban Affairs Library Florida International University Miami, Florida 33199

INSPECTION STATUS

ENFORCEMENT STATUS

NONE.

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

NORMAL OPERATIONS.

LAST IE SITE INSPECTION DATE: SEPTEMBER 9-12, 1980

INSPECTION REPORT NO: 50-251/80-26

SUMMARY

INSPECTION SUMMARY

INSPECTION MAY 5-16 (80-15): THIS SPECIAL ANNOUNCED INSPECTION INVOLVED 234 INSPECTOR-HOURS ON SITE IN THE AREAS OF HEALTH PHYSICS INCLUDING ORGANIZATION, QUALIFICATIONS, TRAINING PROCEDURES, ALARA PROGRAMS, EXTERNAL EXPOSURE CONTROL, PERSONNEL DOSIMETER PROGRAM, INTERNAL DOSIMETRY, RESPIRATORY PROTECTION, MEDICAL EMERGENCIES, INSTRUMENTATION, SURVEILLANCE AND ACCESS CONTROL, RADWASTE CONTROL, FACILITIES AND EQUIPMENT, RE-ENTRY AND IN-PLANT SYSTEMS. OF THE 16 AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN 13 AREAS; THREE ITEMS OF NONCOMPLIANCE WERE FOUND IN THREE AREAS.

INSPECTION JUNE 1-30 (80-26): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 64 INSPECTOR-HOURS ON SITE IN THE AREAS OF FOLLOWUP ON IMPLEMENTATION OF CATEGORY "A" SHORT TERM LESSONS LEARNED, PLANT OPERATIONS, SURVEILLANCE TEST OBSERVATION, AND PLANT TOURS. OF THE FOUR AREAS INSPECTED, NO APPARENT ITEM OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN THREE AREAS; ONE APPARENT ITEMS OF NONCOMPLIANCE WAS FOUND IN ONE AREA (DEFICIENCY - IMPROPER STORAGE OF RADIOACTIVE MATERIAL).

REPORTS RECEIVED FROM LICENSEE

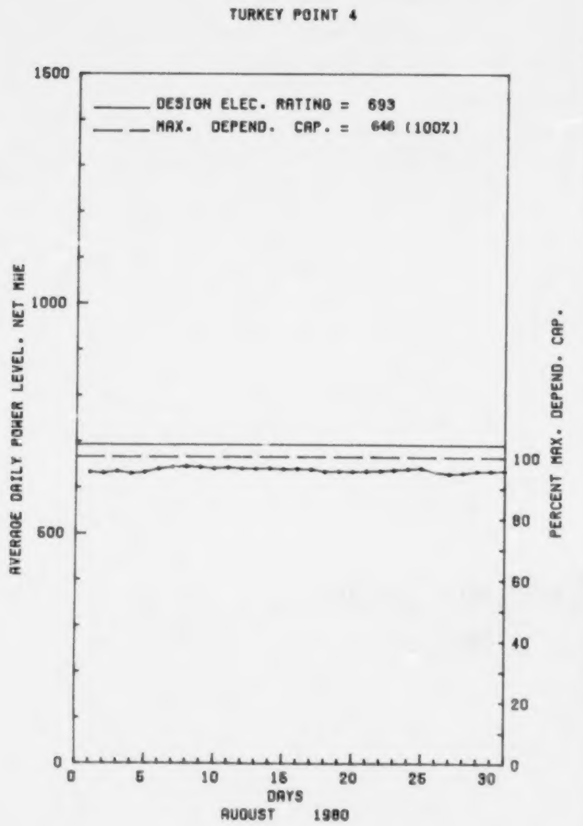
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-009/03L	06/14/80	07/14/80	VALVE STEM SEPARATED FROM DISK PARTIALLY BLOCKING INLET TO RTD BYPASS
80-010/03L	06/17/80	07/14/80	LEAKAGE THRU CORROSION CRACK IN COMPONENT COOLING WATER HEAT EXCHANGER

REVIEWED BY OMPA LOEB: T.C. Cintula **DATE:** 10·6·80

REVIEWED BY NRR DL: M Grotenhuis **DATE:** 10-7-80

TURKEY POINT 4

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-251
2. Utility Contact: V. T. Chilson (305) 552-3824
3. Licensed Thermal Power (MWt): 2200
4. Nameplate Rating (Gross MWe): 894 x 0.85 = 760
5. Design Electrical Rating (Net MWe): 693
6. Maximum Dependable Capacity (Gross MWe): 680
7. Maximum Dependable Capacity (Net MWe): 666

ITEM 21 CALCULATED BY USING A WEIGHTED AVERAGE.

8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: ITEMS 6 & 7 RERATED TO MOST RESTRICTIVE SEASON
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	5,855.0	61,584.0
12. Hours Reactor Was Critical	744.0	4,532.1	45,359.9
13. Reactor Reserve Shutdown Hours	.0	12.3	166.6
14. Hours Generator On-Line	744.0	4,444.8	43,627.3
15. Unit Reserve Shutdown Hours	.0	12.3	31.2
16. Gross Thermal Energy (MWH)	1,633,024	9,629,136	90,848,116
17. Gross Electrical Energy (MWH)	499,282	2,997,355	28,900,833
18. Net Electrical Energy (MWH)	474,126	2,840,239	27,370,235
19. Unit Service Factor	100.0	75.9	70.8
20. Unit Availability Factor	100.0	76.1	70.9
21. Unit Capacity Factor (MDC Net)	98.6	75.1	69.0
22. Unit Capacity Factor (DER Net)	92.0	70.0	64.1
23. Unit Forced Outage Rate	.0	.3	3.0

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUELING, MAIN. & INSPECT. - NOV. 9, 1980 - JAN. 10, 1981.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	NONE								

SUMMARY

TURKEY POINT 4 OPERATED AT APPROXIMATELY FULL POWER FOR THE ENTIRE MONTH WITH NO OUTAGES OR REDUCTIONS.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

VERMONT YANKEE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Vermont/Windham 5 miles S of Brattleboro
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	1593
5.	Design Electrical Rating (MWE Net)	514
6.	Date of Initial Criticality	March 24, 1972
7.	Date of Commercial Operation	November 30, 1972
8.	Condenser Cooling Method	Closed Cycle Cooling Towers
9.	Condenser Cooling Water Source	Connecticut River

Utility & Contractor Information

10.	Licensee	Vermont Yankee Nuclear Power Corp.
11.	Corporate Address	20 Turnpike Road Westboro, Massachusetts 01581
12.	Corporate Contact	D. W. Edwards Assistant to the Vice President
13.	Architect/Engineer	Ebasco
14.	Nuclear Steam Supply System	GE
15.	Constructor	Ebasco

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	IE Section Chief	T. T. MARTIN
19.	IE Resident Inspector	W. J. RAYMOND (802) 257-4319
20.	Licensing Project Manager	V. L. Rooney
21.	Docket Number	50-271
22.	License Number/Date of Issue	DPR-28, 3/21/72
23.	Public Document Room	Brooks Memorial Library 224 Main Street Brattleboro, Vermont 05301

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO TECH SPEC 6.5.B.1 AND AP 0503, ON MAY 5, 1980 TWO ELECTRICAL MAINTENANCE PERSONNEL WERE OBSERVED WORKING INSIDE A HIGH RADIATION AREA WITHOUT RWP CONTROLS ISSUED OR IN EFFECT. (8005 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

FACILITY ITEMS (PLANS AND PROCEDURES):

REFUELING OUTAGE, SCHEDULED TO BEGIN SEPTEMBER 26, 1980. WILL LAST APPROXIMATELY EIGHT WEEKS.

MANAGERIAL ITEMS:

MR. JAMES GRIFFIN, PRESIDENT AND CHIEF EXECUTIVE OFFICER, VERMONT YANKEE, VISITED THE NRC REGION I OFFICE TO MEET WITH THE DIRECTOR ON AUGUST 15, 1980.

MR. MICHAEL LYSTER HAS BEEN PROMOTED TO THE POSITION OF OPERATIONS SUPERVISOR EFFECTIVE SEPTEMBER 1, 1980. MR. ROBERT SOJKA, FORMER OPERATIONS SUPERVISOR, WILL BECOME SPECIAL PROJECTS ENGINEER IN THE WEST BRATTLEBORO OFFICE OF VERMONT YANKEE.

PLANT STATUS:

ROUTINE OPERATIONS WITH POWER OUTPUT LIMITED DUE TO END OF CYCLE COAST DOWN WITH PLANT COOLING TOWERS IN SERVICE. OFFGAS AND STACK RELEASE RATES ARE 2960 UCI/SEC AND LESS THAN 100 UCI/SEC., RESPECTIVELY.

LAST IE SITE INSPECTION DATE: 8/11-15/80

INSPECTION REPORT NO: 50-271/80-12

SUMMARY

INSPECTION SUMMARY

50-271/80-05 - APR 28 - MAY 16: ROUTINE, UNANNOUNCED INSPECTION OF LICENSED ACTIVITIES BY TWO RESIDENT INSPECTORS (92 HRS) INCLUDING: STATUS OF PREVIOUS INSPECTION FINDINGS; PLANT OPERATIONS AND COMPLIANCE WITH TECHNICAL SPECIFICATION REQUIREMENTS; INSPECTOR FOLLOWUP OF EVENTS INVOLVING ACG ANOMALIES AND LOSS OF CONTROL ROOM ALARMS; PHYSICAL SECURITY, SURVEILLANCE TESTING; AND, ADMINISTRATIVE CONTROLS FOR CONTAINMENT PURGING OPERATIONS (TI 2515/26). ONE ITEM OF NONCOMPLIANCE WAS IDENTIFIED: FAILURE TO ESTABLISH RWP CONTROL OVER WORK ACTIVITIES IN A HIGH RADIATION AREA.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

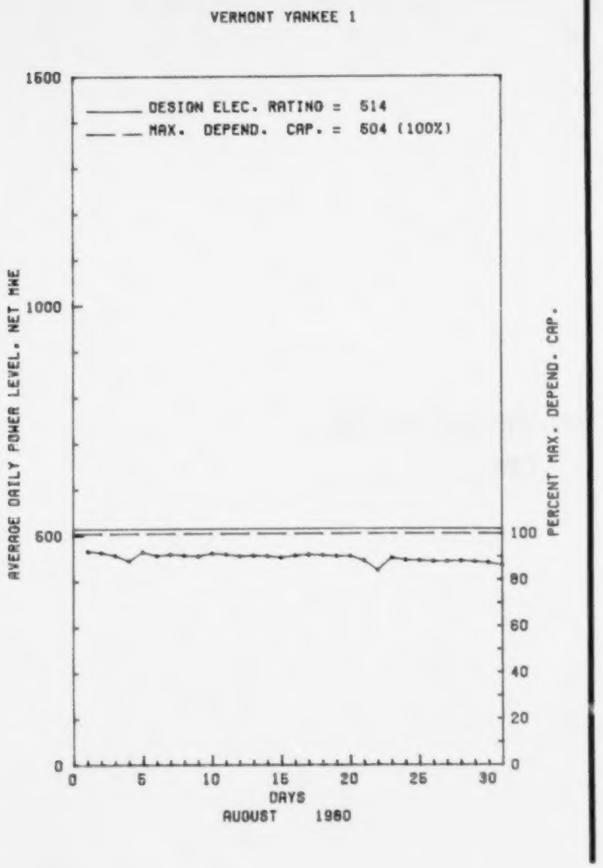
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-020/ 03L	06/17/80	07/17/80	FAILURE OF CONTAINMENT ISOL. VALVE. (FSO-109-76A) TO CLOSE FOLLOWING SCRAM DUE TO DIRT ON PLUNGER ASSEMBLY.
80-021/ 03L	06/30/80	07/31/80	WEEKLY ENVIRONMENTAL AIR SAMPLES NOT COLLECTED AT THREE STATIONS DUE TO FAILED SAMPLE PUMPS.
80-022/ 03L	07/09/80	08/06/80	DRYWELL/TORUS D/P REDUCED DUE TO FAILURE OF VACUUM BKR TO SEAT.
80-023/ 03L	07/12/80	08/06/80	B RECIRC PUMP MOTOR SNUBBER FAILURE DUE TO FLUID LKG AT LENZ TUBING CONNECTION.
80-024/ 03L	07/18/80	08/15/80	LOGIC POWER SUPPLY BOARD FOR UPS-1B FAILED.
80-025/ 03L	07/21/80	08/20/80	MS-DPIS-2-120C (MS LINE HIGH FLOW D/P) SETPOINT DRIFT HIGH.

REVIEWED BY OMPA LOEB: John L Crooks **DATE:** 10/6/80

REVIEWED BY NRR DL: **DATE:** 10/1/80

VERMONT YANKEE

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-271
2. Utility Contact: F.C. BEERS (617) 366-9011
3. Licensed Thermal Power (MWt): 1593
4. Nameplate Rating (Gross MWe): 626 x 0.9 = 563
5. Design Electrical Rating (Net MWe): 514
6. Maximum Dependable Capacity (Gross MWe): 535
7. Maximum Dependable Capacity (Net MWe): 504
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	5,855.0	69,649.8
12. Hours Reactor Was Critical	744.0	5,642.2	56,800.8
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	744.0	5,576.6	54,904.4
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,098,752	8,478,526	77,703,137
17. Gross Electrical Energy (MWH)	359,195	2,849,963	25,819,633
18. Net Electrical Energy (MWH)	335,949	2,705,993	24,474,358
19. Unit Service Factor	100.0	95.2	78.8
20. Unit Availability Factor	100.0	95.2	78.8
21. Unit Capacity Factor (MDC Net)	89.6	91.7	69.7
22. Unit Capacity Factor (DER Net)	87.8	89.9	68.4
23. Unit Forced Outage Rate	.0	2.1	6.1

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
REFUELING - 09/26/80 - 8 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-20	08/04/80	S	.0	B	5		CB	PUMPXX	POWER REDUCTION FOR DRYWELL ENTRY TO CHECK RECIRC PUMP MOTOR COOLING SYSTEM.

SUMMARY

VERMONT YANKEE OPERATED AT 90% POWER FOR THE ENTIRE MONTH, WITH ONE REDUCTION TO CHECK RECIRCULATING PUMP MOTOR COOLING SYSTEM.

LEGEND

[1] Type:
F–Forced
S–Scheduled

[2] Reason:
A–Equipment Failure
B–Maintenance or Test
C–Refueling
D–Regulatory Restriction
E–Operator Training & License Examination
F–Administrative
G–Operational Error
H–Other

[3] Method:
1–Manual
2–Manual Scram
3–Automatic Scram
4–Continuations
5–Load Reduction
9–Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

YANKEE ROWE

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) ~~(nearest population center)~~	Massachusetts/Franklin 25 miles NE of Pittsfield
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	600
5.	Design Electrical Rating (MWE Net)	175
6.	Date of Initial Criticality	August 19, 1960
7.	Date of Commercial Operation	July 1961
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Deerfield River

Utility & Contractor Information

10.	Licensee	Yankee Atomic Electric Company
11.	Corporate Address	20 Turnpike Road Westboro, Massachusetts 01581
12.	Corporate Contact	D. W. Edwards, Asst. to V.P.
13.	Architect/Engineer	Stone & Webster
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Section Chief	T. T. MARTIN
19.	IE Resident Inspector	T. Foley (413) 424-5242
20.	Licensing Project Manager	R. Caruso (X27232)
21.	Docket Number	50-29
22.	License Number/Date of Issue	DPR-3, 12/24/63
23.	Public Document Room	Greenfield Community College 1 College Drive Greenfield, Massachusetts 01301

INSPECTION STATUS

ENFORCEMENT STATUS

SECURITY GUARD WAS NOT STATIONED AT THE POINT OF ACCESS TO A CONTROLLED ACCESS AREA AND DID NOT CONTROL OR RECORD ACCESS TO THE NEW FUEL VAULT, AN ACCESS CONTROLLED AREA. THIS IS CONTRARY TO YNPS DP0475 AND T/S 6.8.1. THIS IS AN INFRACTION.
(8002 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

MANAGERIAL ITEMS:

D. CRUTCHFIELD OF NRR, AND J. TRIBLE, PRESIDENT OF YAEC, TOURED THE YANKEE ROWE SITE ON AUGUST 5, 1980, AS PART OF AN NRR TOUR OF PLANTS LOCATED IN REGION I.

PLANT STATUS:

PLANT REMAINS IN COLD SHUTDOWN.

NEW TURBINE BLADES HAVE ARRIVED ON SITE. REBLADING OF THE L.P. TURBINE COMMENCED IN AUGUST.

LAST IE SITE INSPECTION DATE: 8/2-30/80

INSPECTION REPORT NO: 50-29/80-14

SUMMARY

INSPECTION SUMMARY

50-29/80-08 - MAY 5-16: ROUTINE, UNANNOUNCED INSPECTION OF PLANT OPERATIONS BY ONE REGION-BASED INSPECTOR (76 HRS) INCLUDING: LOG AND RECORD REVIEW; TOUR OF THE FACILITY; OBSERVATION OF PHYSICAL SECURITY; HOUSEKEEPING; RADIATION PROTECTION CONTROLS; FOLLOWUP OF SELECTED LICENSEE EVENT REPORTS (LERS); AND, FOLLOWUP OF SELECTED IE BULLETINS. ONE ITEM OF NONCOMPLIANCE WAS IDENTIFIED: FAILURE TO CONTROL ENTRY TO AN ACCESS CONTROLLED AREA.

50-29/80-09 - JUN 26: MANAGEMENT MEETING HELD AT NRC'S REQUEST TO DISCUSS THE REGULATORY PERFORMANCE OF YANKEE NUCLEAR POWER STATION AS CONCLUDED IN THE SYSTEMATIC ASSESSMENT OF LICENSEE PERFORMANCE (SALP) PROGRAM. A SUMMARY OF THE LICENSEE PERFORMANCE EVALUATION WAS PRESENTED. PERFORMANCE WAS CONSIDERED TO BE ACCEPTABLE.

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
None.			

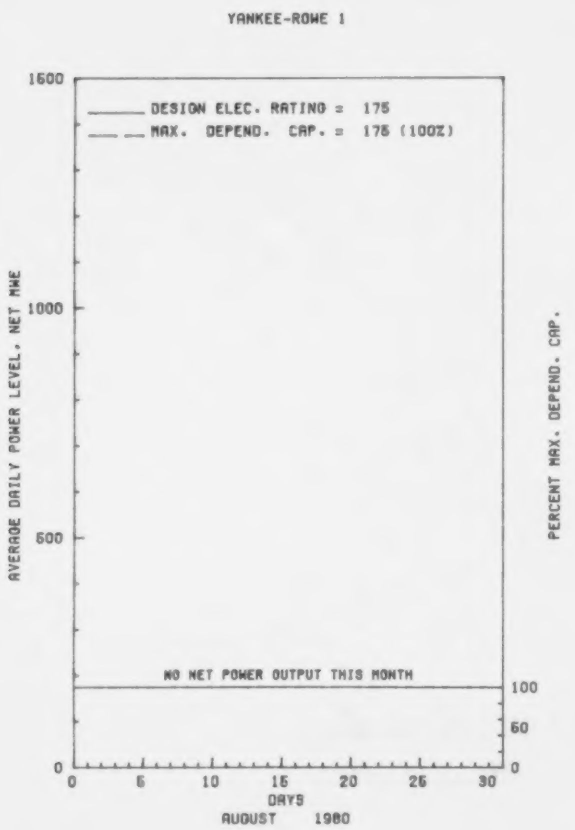
REVIEWED BY OMPA LOEB: T.C. CINTULA **DATE:** 10·6·80

REVIEWED BY NRR DL: **DATE:** 10/8/80

Y-2

YANKEE ROWE

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-029
2. Utility Contact: F.C. BEERS (617) 366-9011
3. Licensed Thermal Power (MWt): 600
4. Nameplate Rating (Gross MWe): 185 x 1.0 = 185
5. Design Electrical Rating (Net MWe): 175
6. Maximum Dependable Capacity (Gross MWe): 180
7. Maximum Dependable Capacity (Net MWe): 175

CAPACITY FACTORS CALCULATED WITH A WEIGHTED AVERAGE

8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	5,855.0	173,492.0
12. Hours Reactor Was Critical	.0	445.8	138,810.4
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	.0	439.7	134,467.1
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	253,769	72,902,091
17. Gross Electrical Energy (MWH)	0	80,635	22,152,788
18. Net Electrical Energy (MWH)	0	75,749	20,761,893
19. Unit Service Factor	.0	7.5	77.5
20. Unit Availability Factor	.0	7.5	77.5
21. Unit Capacity Factor (MDC Net)	.0	7.4	70.5
22. Unit Capacity Factor (DER Net)	.0	7.4	70.5
23. Unit Forced Outage Rate	100.0	90.2	4.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 11/01/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	01/19/80	F	744.0	A	4				THE PLANT REMAINS SHUTDOWN FOR TURBINE ROTOR INVESTIGATION AND REPAIR.

SUMMARY

THE PLANT REMAINS SHUTDOWN FOR TURBINE ROTOR INVESTIGATION AND REPAIR.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

ZION 1

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Illinois/Lake 40 miles N of Chicago
2.	Electric Reliability Council	Mid-America Interpool Network
3.	Type of Reactor	PWR
4.	Power Level (MWT)	3250
5.	Design Electrical Rating (MWE Net)	1040
6.	Date of Initial Criticality	June 19, 1973
7.	Date of Commercial Operation	December 31, 1973
8.	Condenser Cooling Method	Straight Through
9.	Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10.	Licensee	Commonwealth Edison Co.
11.	Corporate Address	P. O. Box 767 Chicago, Illinois
12.	Corporate Contact	Cordell Reed, Manager of Nuclear Licensing
13.	Architect/Engineer	Sergent & Lundy
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Various

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	R. L. Spessard
19.	IE Resident Inspector	J. E. Kohler (312) 746-2313
20.	Licensing Project Manager	D. Wigginton
21.	Docket Number	50-295
22.	License Number/Date of Issue	DPR-39 issued 4/6/73
23.	Public Document Room	Zion-Benton Public Library 2600 Emmans Avenue Zion, Illinois 60099

INSPECTION STATUS

ENFORCEMENT STATUS

10 CFR 50.72(A)(8) REQUIRES NUCLEAR POWER PLANT LICENSEES TO NOTIFY THE NRC OPERATIONS CENTER WITHIN ONE HOUR OF ANY ACCIDENTAL, UNPLANNED OR UNCONTROLLED RADIATION RELEASE. CONTRARY TO THE ABOVE, THE LICENSEE FAILED TO REPORT WITHIN ONE HOUR AN ACCIDENTAL, UNPLANNED GASEOUS RELEASE EQUIVALENT TO APPROXIMATELY 12% OF THE TECHNICAL SPECIFICATION LIMIT FOR INSTANTANEOUS RELEASES.
(8012 DEFICIENCY)

TECHNICAL SPECIFICATION 3.12.1.C.4.2 STATES THAT RADIOACTIVE GASEOUS WASTES COLLECTED IN THE GAS DECAY TANKS SHALL BE HELD UP TO A MINIMUM OF 45 DAYS BEFORE BEING RELEASED EXCEPT DURING PURGE AND FILL OPERATIONS ASSOCIATED WITH REFUELING AND REACTOR STARTUP. CONTRARY TO THE ABOVE, ON MAY 6, 1980, A PORTION OF THE CONTENTS OF GAS DECAY TANK 1B WAS RELEASED WITHOUT BEING HELD FOR 45 DAYS.
(8012 INFRACTION)

CODE OF FEDERAL REGULATIONS SECTION 173.393(J)(2) LIMITS TO 200 MR/HR THE EXPOSURE RATE AT THE EXTERNAL SURFACE OF THE VEHICLE. CONTRARY TO THE ABOVE, ON FEBRUARY 22 AND 26, 1980, AN EXPOSURE RATE OF 220 MR/HR WAS MEASURED AT A SIDE SURFACE OF HOME TRANSPORTATION COMPANY INCORPORATED TRAILER 442054, WHICH CONTAINED 70 55-GALLON DRUMS OF WASTE SHIPPED ON FEBRUARY 17, 1980, FROM ZION TO NUCLEAR ENGINEERING COMPANY; RICHLAND, WASHINGTON.
(8013 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

MANAGERIAL ITEMS

Mr. K. Graesser assumed the duties of Superintendent and Mr. Kurt Kofron assumed the duties of Assistant Superintendent - Maintenance. (80-14)

PLANT STATUS

Unit 1 operating routinely.

LAST IE SITE INSPECTION DATE: July 30-31, 1980

INSPECTION REPORT NO: 80-18

SUMMARY

INSPECTION SUMMARY

Inspection on February 22 and 26, (80-13): This report details the inspection of Trailer 442054 which was part of the inspection conducted February 19-29, 1980, at NECO, Richland, Washington. A State of Washington representative was onsite. One drum was opened on February 26, 1980, by Zion Station personnel with approval of Washington State representatives. Maximum dose rate on the surface of the transport trailer was 220 mR/hr. This was in noncompliance with DOT regulations. It appears that the Department of Transportation and Region III are the appropriate organizations to take enforcement actions.

Inspection on June 1 through July 31, (80-14): Routine, unannounced resident inspection of containment purging, regulatory guide 1.108, diesel generator testing, TMI mods, licensing administration, containment integrated leak rate testing, auxiliary feedwater pump requirements, violation of low pressure over pressure protection, event V check valves, steam generator snubbers, damage to 1A charging pump, steam generator tube rupture procedure, W-2 switches, inadvertent boron dilution at cold shutdown, unresolved items, violation of dilution requirements, operational safety, maintenance, surveillance, LER's, IE Bulletin followup, refueling activities, maintenance during refueling, retests and procedure changes, refueling plant operations, spent fuel storage racks, and onsite review committee. The inspection involved 442 inspector-hours onsite by three NRC inspectors including 51 inspector-hours during off-shifts. Of the areas inspected, no items of noncompliance were identified.

Inspection on July 22, (80-15): Review of specifications and procedures, observation of work, and review of documentation for installation of spent fuel storage racks; followup on IE Bulletin 80-08 "Examination of Containment Liner Penetration Welds." The inspection involved eight inspector-hours onsite by one NRC inspector. No items of noncompliance or deviations were identified.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-26/ 03L-0	07/15/80	08/14/80	Class I Pipe Tunnel Iodine Monitor IRT-PR07B failed low during normal operation.
80-31/ 03L-0	07/18/80	08/15/80	The auxiliary building area monitor ORT-AR09 failed low during normal operation.
80-35/ 01T-0	07/20/80	07/30/80	The pH of the WTF effluent which was discharged to the condenser cooling water exceeded ETS limits.
80-36/ 03L-0	07/11/80	08/08/80	Pzr. level channel 1L-460 drifted low by 4% during operation.

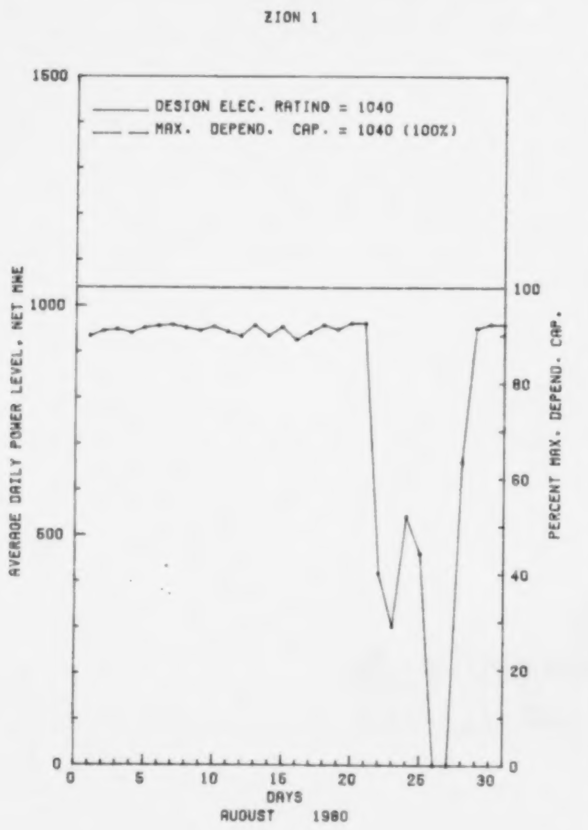
Inspection on July 30-31, (80-18): Included a review of the Protected Area Physical Barriers; Vital Area Physical Barriers and Licensee Action Taken on Previous Inspection Findings. The inspection involved 23 inspector-hours onsite by two NRC inspectors. The inspection began during the day shift; two inspector-hours were accomplished during off-shift periods. The licensee was found to be in compliance with NRC requirements within the areas examined.

REVIEWED BY OMPA LOEB: T.C. Cintula **DATE:** 10.6.80

REVIEWED BY NRR DL: DLWigginton **DATE:** 10.7.80

ZION 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-295
2. Utility Contact: J. COOK (312) 746-2084
3. Licensed Thermal Power (MWt): 3250
4. Nameplate Rating (Gross MWe): 1220 x 0.9 = 1098
5. Design Electrical Rating (Net MWe): 1040
6. Maximum Dependable Capacity (Gross MWe): 1085
7. Maximum Dependable Capacity (Net MWe): 1040
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	5,855.0	58,463.0
12. Hours Reactor Was Critical	719.2	4,641.5	41,749.2
13. Reactor Reserve Shutdown Hours	.0	.0	2,621.8
14. Hours Generator On-Line	668.9	4,489.5	40,576.8
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	2,048,865	13,853,092	113,236,093
17. Gross Electrical Energy (MWH)	639,670	4,297,150	36,529,200
18. Net Electrical Energy (MWH)	602,508	4,086,457	34,570,703
19. Unit Service Factor	89.9	76.7	69.4
20. Unit Availability Factor	89.9	76.7	69.4
21. Unit Capacity Factor (MDC Net)	77.9	67.1	56.9
22. Unit Capacity Factor (DER Net)	77.9	67.1	56.9
23. Unit Forced Outage Rate	10.1	23.3	14.4

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
REFUELING - JANUARY 1, 1981 - APPROXIMATELY 7 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
16	08/22/80	F	19.1	B	3				REACTOR TRIP DUE TO 1B STEAM GENERATOR LO-LO LEVEL.
17	08/26/80	F	44.3	A	1				GENERATOR OFF-LINE DUE TO VOLTAGE REGULATION PROBLEM.
18	08/27/80	F	11.7	A	3				REACTOR TRIP/TURBINE TRIP DUE TO 1B FWP.

SUMMARY

ZION 1 OPERATED AT APPROXIMATELY 90% POWER WITH 3 OUTAGES DUE TO TURBINE AND GENERATOR PROBLEMS.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

ZION 2

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Illinois/Lake 40 miles N of Chicago
2.	Electric Reliability Council	Mid-America Interpool Network
3.	Type of Reactor	PWR
4.	Licenses Power Level (MWT)	3250
5.	Design Electrical Rating (MWE Net)	1040
6.	Date of Initial Criticality	December 24, 1973
7.	Date of Commercial Operation	September 17, 1974
8.	Condenser Cooling Method	Straight Through
9.	Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10.	Licensee	Commonwealth Edison Co.
11.	Corporate Address	P. O. Box 767 Chicago, Illinois
12.	Corporate Contact	Cordell Reed, Manager of Nuclear Licensing
13.	Architect/Engineer	Sargent & Lundy
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Various

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	R. L. Spessard
19.	IE Resident Inspector	J. E. Kohler (312) 746-2313
20.	Licensing Project Manager	D. Wigginton
21.	Docket Number	50-304
22.	License Number/Date of Issue	DPR-48, 11/14/73
23.	Public Document Room	Zion-Benton Public Library 2600 Emmans Avenue Zion, Illinois 60099

INSPECTION STATUS

ENFORCEMENT STATUS

10 CFR 50.72(A)(8) REQUIRES NUCLEAR POWER PLANT LICENSEES TO NOTIFY THE NRC OPERATIONS CENTER WITHIN ONE HOUR OF ANY ACCIDENTAL, UNPLANNED OR UNCONTROLLED RADIATION RELEASE. CONTRARY TO THE ABOVE, THE LICENSEE FAILED TO REPORT WITHIN ONE HOUR AN ACCIDENTAL, UNPLANNED GASEOUS RELEASE EQUIVALENT TO APPROXIMATELY 12% OF THE TECHNICAL SPECIFICATION LIMIT FOR INSTANTANEOUS RELEASES.
(8012 DEFICIENCY)

TECHNICAL SPECIFICATION 3.12.1.C.4.2 STATES THAT RADIOACTIVE GASEOUS WASTES COLLECTED IN THE GAS DECAY TANKS SHALL BE HELD UP TO A MINIMUM OF 45 DAYS BEFORE BEING RELEASED EXCEPT DURING PURGE AND FILL OPERATIONS ASSOCIATED WITH REFUELING AND REACTOR STARTUP. CONTRARY TO THE ABOVE, ON MAY 6, 1980, A PORTION OF THE CONTENTS OF GAS DECAY TANK 1B WAS RELEASED WITHOUT BEING HELD FOR 45 DAYS.
(8012 INFRACTION)

CODE OF FEDERAL REGULATIONS SECTION 173.393(J)(2) LIMITS TO 200 MR/HR THE EXPOSURE RATE AT THE EXTERNAL SURFACE OF THE VEHICLE. CONTRARY TO THE ABOVE, ON FEBRUARY 22 AND 26, 1980, AN EXPOSURE RATE OF 220 MR/HR WAS MEASURED AT A SIDE SURFACE OF HOME TRANSPORTATION COMPANY INCORPORATED TRAILER 442054, WHICH CONTAINED 70 55-GALLON DRUMS OF WASTE SHIPPED ON FEBRUARY 17, 1980, FROM ZION TO NUCLEAR ENGINEERING COMPANY; RICHLAND, WASHINGTON.
(8013 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENTS

TMI head vent and vessel water level installed on Zion Unit 2. (80-14)

FACILITY ITEMS

Zion Unit 2 returned to service on July 26, 1980, following a refueling and maintenance outage which lasted 85 days. (80-14)

MANAGERIAL ITEMS

Mr. K. Graesser assumed the duties of Superintendent and Mr. Kurt Kofron assumed the duties of Assistant Superintendent - Maintenance. (80-14)

PLANT STATUS

Unit 2 operating routinely.

LAST IE SITE INSPECTION DATE: July 30-31, 1980

INSPECTION REPORT NO: 80-18

SUMMARY

INSPECTION SUMMARY

Inspection on February 22 and 26, (80-13): This report details the inspection of Trailer 442054 which was part of the inspection conducted February 19-29, 1980, at NECO, Richland, Washington. A State of Washington representative was onsite. One drum was opened on February 26, 1980, by Zion Station personnel with approval of Washington State representatives. Maximum dose rate on the surface of the transport trailer was 220 mR/hr. This was in noncompliance with DOT regulations. It appears that the Department of Transportation and Region III are the appropriate organizations to take enforcement actions.

Inspection on June 1 through July 31, (80-14): Routine, unannounced resident inspection of containment purging, regulatory guide 1.108, diesel generator testing, TMI mods, licensing administration, containment integrated leak rate testing, auxiliary feedwater pump requirements, violation of low pressure over pressure protection, event V check valves, steam generator snubbers, damage to 1A charging pump, steam generator tube rupture procedure, W-2 switches, inadvertent boron dilution at cold shutdown, unresolved items, Unit 2 return to service, violation of dilution requirements, operational safety, maintenance, surveillance, LER's, IE Bulletin followup, refueling activities, maintenance during refueling, retests and procedure changes, refueling plant operations, spent fuel storage racks, and onsite review committee. The inspection involved 442 inspector-hours onsite by three NRC inspectors including 51 inspector-hours during off-shifts. Of the areas inspected, one item of noncompliance (violation of discharge dilution requirements) was identified

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-07/ 01T-0	08/03/80	08/15/80	During check of Containment Isolation phase A final actuating relays, the 1W partially actuated giving a low volume and low pressure.

Inspection on July 22, (80-15): Review of specifications and procedures, observation of work, and review of documentation for installation of spent fuel storage racks; followup on IE Bulletin 80-08 "Examination of Containment Liner Penetration Welds." The inspection involved eight inspector-hours onsite by one NRC inspector. No items of noncompliance or deviations were identified.

Inspection on August 4-5, (80-16): Routine, announced inspection of the licensed operator requalification training program. The inspection involved 12 inspector-hours onsite by one NRC inspector. One item of noncompliance was identified (failure to follow requalification program procedures).

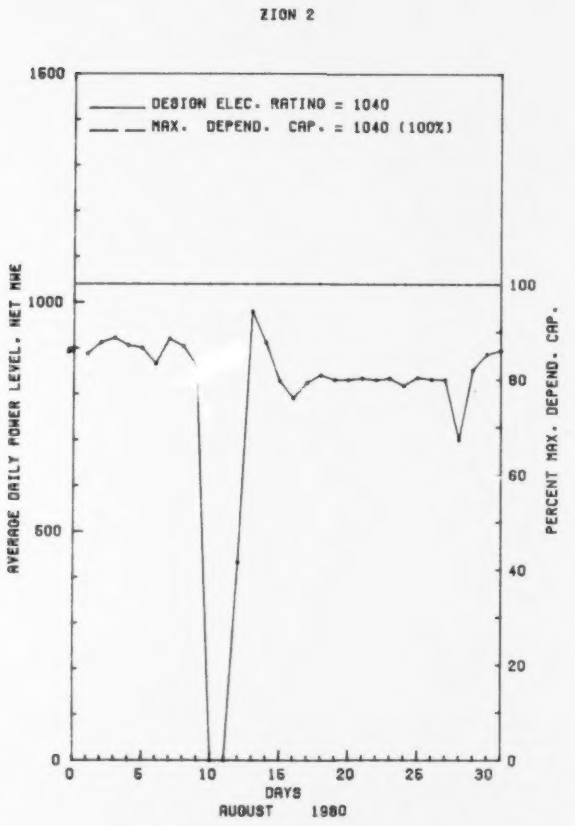
Inspection on July 30-31, (80-18): Included a review of the Protected Area Physical Barriers; Vital Area Physical Barriers and Licensee Action Taken on Previous Inspection Findings. The inspection involved 23 inspector-hours onsite by two NRC inspectors. The inspection began during the day shift; two inspector-hours were accomplished during off-shift periods. The licensee was found to be in compliance with NRC requirements within the areas examined.

REVIEWED BY OMPA LOEB: T.C. Cintula **DATE:** 10·6·80

REVIEWED BY NRR DL: DWigginton **DATE:** 10·7·80

ZION 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-304
2. Utility Contact: J. COOK (312) 746-2084
3. Licensed Thermal Power (MWt): 3250
4. Nameplate Rating (Gross MWe): 1220 x 0.9 = 1098
5. Design Electrical Rating (Net MWe): 1040
6. Maximum Dependable Capacity (Gross MWe): 1085
7. Maximum Dependable Capacity (Net MWe): 1040
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	744.0	5,855.0	52,176.0
12. Hours Reactor Was Critical	734.5	3,368.9	36,972.5
13. Reactor Reserve Shutdown Hours	.0	.0	226.1
14. Hours Generator On-Line	686.6	3,197.2	36,013.1
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,959,457	9,566,242	102,201,560
17. Gross Electrical Energy (MWH)	618,835	3,107,280	32,828,500
18. Net Electrical Energy (MWH)	586,447	2,937,289	31,138,747
19. Unit Service Factor	92.3	54.6	69.0
20. Unit Availability Factor	92.3	54.6	69.0
21. Unit Capacity Factor (MDC Net)	75.8	48.2	57.4
22. Unit Capacity Factor (DER Net)	75.8	48.2	57.4
23. Unit Forced Outage Rate	7.7	21.3	18.0

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
18	08/09/80	F	15.9	B	1				TURBINE/GENERATOR OFF FOR REPAIRS OF STATOR WATER COOLING PUMPS.
19	08/10/80	F	5.1	A	3				REACTOR TRIP DUE TO STEAM GENERATOR 2D LO-LEVEL IN COINCIDENT WITH STEAM FLOW/FEED FLOW MISMATCH DUE TO STEAM SPIKE WHILE ATTEMPTING TO START B FEEDWATER PUMP.
20	08/10/80	F	36.4	A	3				REACTOR TRIP STEAM GENERATOR 2C STEAM FLOW/FEED FLOW MISMATCH COINCIDENT WITH LO-LEVEL CAUSED BY GENERATOR REVERSE POWER TRIP DUE TO A PROBLEM WITH THE EHC SYSTEM.
21	08/28/80	F	.0	A	5				LOAD REDUCED TO LOCATE PRIMARY COOLANT SYSTEM LEAK.

SUMMARY

ZION 2 OPERATED AT APPROXIMATELY 85% POWER WITH THREE OUTAGES RELATED TO TURBINE/GENERATOR PROBLEMS AND ONE REDUCTION TO LOCATE A PRIMARY COOLANT SYSTEM LEAK.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

SECTION 3 – APPENDIX

STATUS OF SPENT FUEL STORAGE CAPABILITY

DATA AS OF 08/31/80

Facility	Core Size [a] (No. of Assemblies)	Present Authorized Storage Pool Capacity (Fuel Assemblies)	No. of Assemblies Stored	Remaining Capacity (No. of Assemblies)	Remaining Capacity If Pending Request Approved (No. of Assemblies)	Scheduled Date For Next Refueling	Will Fill Present [b] Authorized Capacity
I. Pressurized Water Reactors							
ARKANSAS 1	177	590	176	414		01-81	1986
ARKANSAS 2	177	486	58	428		03-81	1989
BEAVER VALLEY 1	157	833	0	833		N/S	1995
CALVERT CLIFFS 1	217	1056 (c)	364 (c)	692 (c)		10-80	1984
CALVERT CLIFFS 2	217					01-81	
COOK 1	193	500	193	307	1857	N/S	1985
COOK 2	193					N/S	
CRYSTAL RIVER 3	177	256	112	144	1109	N/S	1982
DAVIS-BESSE 1	177	735	52	683		03-82	1989
FARLEY 1	157	675	0	675		09-80	1990
FORT CALHOUN 1	133	483	197	286		03-81	1985
GINNA	121	595	156	439		N/S	1987
HADDAM NECK	157	1168	389	779		09-81	1994
INDIAN POINT 1	0	288	160	128		N/S	
INDIAN POINT 2	193	482	200	282		N/S	1984
INDIAN POINT 3	193	837	64	773		N/S	1989
KEWAUNEE	121	990	120	870		N/S	1995
MAINE YANKEE	217	953	433	520		N/S	1986
MILLSTONE 2	217	667	144	523		N/S	1987
NORTH ANNA 1	157	400	0	400	966	N/S	1984
NORTH ANNA 2(d)	157	400	0	400			
OCONEE 1	177	1224 (l)	737	487 (l)	755	05-81	1980
OCONEE 2	177					06-81	
OCONEE 3	177					12-80	
PALISADES	204	798	273	525		N/S	1985
POINT BEACH 1	121	1502 (c)	276 (c)	1226 (c)		11-80	1995
POINT BEACH 2	121					03-81	
PRAIRIE ISLAND 1	121	687 (c)	280 (c)	407 (c)		09-80	1984
PRAIRIE ISLAND 2	121					01-81	
RANCHO SECO 1	177	579	164	415		04-81	1987
ROBINSON 2	157	272	106	166 (e)		N/S	1980 (g)
SALEM 1	193	264	40	224	1130	09-80	1982
SAN ONOFRE 1	157	216	97	119		N/S	1983
ST LUCIE 1	217	728	60	668		N/S	1990
SURRY 1	157	1044 (c)	1983 (c)	608 (c)		09-80	1983
SURRY 2	157					09-80	
THREE MILE ISLAND 1	177	752	208	544		N/S	1986
THREE MILE ISLAND 2	177	442	0	442		N/S	1986
TROJAN	193	651	126	587		04-81	1988
TURKEY POINT 3	157	621	346 (c)	275 (c)		N/S	1987
TURKEY POINT 4	157					N/S	
YANKEE-ROWE 1	76	391	149	242	572	N/S	1985
ZION 1	193	2112 (c)	508 (c)	1604 (c)	1676	01-81	1993
ZION 2	193					N/S	

Facility	Core Size [a] (No. of Assemblies)	Present Authorized Storage Pool Capacity (Fuel Assemblies)	No. of Assemblies Stored	Remaining Capacity (No. of Assemblies)	Remaining Capacity If Pending Request Approved (No. of Assemblies)	Scheduled Date For Next Refueling	Will Fill Present [b] Authorized Capacity
II. Boiling Water Reactors							
BROWNS FERRY 1	764	3471	556	2795		04-81	1985
BROWNS FERRY 2	764	1080	640	440		09-80	1985
BROWNS FERRY 3	764	1528	516	1012		10-80	1985
BRUNSWICK 1	560	(f)	154PWR+476BWR			N/S	1986
BRUNSWICK 2	560		105PWR+264BWR			N/S	1986
COOPER STATION	548	2366	284	2082		N/S	1995
DRESDEN 1	464	672	221	451		N/S	1990
DRESDEN 2	724	2840 (c)	1069 (c)	1771 (c)	6491 (c)	12-80	1985
DRESDEN 3	724					N/S	
DUANE ARNOLD	368	2050	364	1686		04-81	1998
FITZPATRICK	560	760	428	492	1816	09-81	1981
HATCH 1	560	840	260	580		12-80	1988
HATCH 2	560	1120	0	1120		10-80	
HUMBOLDT BAY	172	487	251	236		N/S	
MILLSTONE 1	580	2184	776	1408		09-80	1991
MONTICELLO	484	2237	828	1409		03-81	1991
NINE MILE POINT 1	532	1984	660	1324	2349	N/S	1990
OYSTER CREEK 1	560	1800	780	1020		N/S	1987
PEACH BOTTOM 2	764	2816	910	1906		01-82	1990
PEACH BOTTOM 3	764	2816	712	2104		03-81	1991
PILGRIM 1	580	2320	764	1556		09-81	1990
QUAD CITIES 1	724	1460	596	864		09-80	1986
QUAD CITIES 2	724	1460	492	968		N/S	1985
VERMONT YANKEE 1	368	2000	894	1106		09-80	1988
BIG ROCK POINT 1	84	193	88	105	343	10-80	1983
LA CROSSE	72	134	113	21	327	11-80	NOW
III. Independent Spent Fuel Storage Installations (h)							
Morris Operations		750 MTU (j)	315	385 MTU (j)	1490 MTU (j)		
NFS(i)		250 MTU	170 MTU	80 MTU			

(a) At each refueling outage approximately 1/3 of a PWR core and 1/4 of a BWR core is off-loaded.
(b) Some of these dates have been adjusted by staff assumptions.
(c) This is the total for both units.
(d) Plant not in commercial operation.
(e) Some spent fuel stored at Brunswick.
(f) Authorized a total 2088 BWR and 304 PWR assemblies for both pools.
(g) Robinson 2 assemblies being shipped to Brunswick for storage.
(h) Capacity is in metric tons of uranium; 1 MTU = 2 PWR assemblies or 5 BWR assemblies.
(i) No longer accepting spent fuel.
(j) Racked for 700 MTU.
(k) Reserved
(l) This is the station total.

N/S = Not Scheduled

REACTOR-YEARS OF EXPERIENCE

08/31/80

The total of Reactor years of experience is as the sum of all calendar days for each unit, from the date that electricity was first generated until a final shutdown date or the status date, whichever comes first, divided by 365.25 days/year.

If a date is unknown, the first day of the first month of operation is substituted. Units which have not yet generated electricity but which are licensed are listed but not included in the computation.

LICENSED OPERATING ELECTRICAL PRODUCING UNITS

UNIT NAME	FIRST ELEC. GENERATION	YEARS EXPERIENCE
ARKANSAS 1	08/01/74	6.09
ARKANSAS 2	12/26/78	1.68
BEAVER VALLEY 1	06/14/76	4.22
BIG ROCK POINT 1	12/08/62	17.73
BROWNS FERRY 1	10/15/73	6.88
BROWNS FERRY 2	08/28/74	6.01
BROWNS FERRY 3	09/12/76	3.97
BRUNSWICK 1	12/04/76	3.74
BRUNSWICK 2	04/29/75	5.34
CALVERT CLIFFS 1	01/03/75	5.66
CALVERT CLIFFS 2	12/07/76	3.73
COOK 1	02/10/75	5.56
COOK 2	03/22/78	2.45
COOPER STATION	05/10/74	6.31
CRYSTAL RIVER 3	01/30/77	3.59
DAVIS-BESSE 1	08/28/77	3.01
DRESDEN 2	04/13/70	10.39
DRESDEN 3	07/22/71	9.11
DUANE ARNOLD	05/19/74	6.29
FARLEY 1	08/18/77	3.04
FITZPATRICK	02/01/75	5.58
FORT CALHOUN 1	08/25/73	7.02
FORT ST VRAIN	12/11/76	3.72
GINNA	12/02/69	10.75
HADDAM NECK	08/07/67	13.07
HATCH 1	11/11/74	5.81
HATCH 2	09/22/78	1.94
INDIAN POINT 2	06/26/73	7.18
INDIAN POINT 3	04/27/76	4.35
KEWAUNEE	04/08/74	6.40
LA CROSSE	04/26/68	12.35
MAINE YANKEE	11/08/72	7.81
MILLSTONE 1	11/29/70	9.76
MILLSTONE 2	11/09/75	4.81
MONTICELLO	03/05/71	9.49
NINE MILE POINT 1	11/09/69	10.81
NORTH ANNA 1	04/17/78	2.38
NORTH ANNA 2	08/25/80	.02
OCONEE 1	05/06/73	7.32
OCONEE 2	12/05/73	6.74
OCONEE 3	09/01/74	6.00
OYSTER CREEK 1	09/23/69	10.94
PALISADES	12/31/71	8.67
PEACH BOTTOM 2	02/18/74	6.54
PEACH BOTTOM 3	09/01/74	6.00
PILGRIM 1	07/19/72	8.12
POINT BEACH 1	11/06/70	9.82
POINT BEACH 2	08/02/72	8.08
PRAIRIE ISLAND 1	12/04/73	6.74
PRAIRIE ISLAND 2	12/21/74	5.70
QUAD CITIES 1	04/12/72	8.39
QUAD CITIES 2	05/23/72	8.28
RANCHO SECO 1	10/13/74	5.89
ROBINSON 2	09/26/70	9.93
SALEM 1	12/25/76	3.69

LICENSED OPERATING ELECTRICAL PRODUCING UNITS - CONT.

UNIT NAME	FIRST ELEC. GENERATION	YEARS EXPERIENCE
SAN ONOFRE 1	07/16/67	13.13
ST LUCIE 1	05/07/76	4.32
SEQUOYAH	07/22/80	0.11
SURRY 1	07/04/72	8.16
SURRY 2	03/10/73	7.48
THREE MILE ISLAND 1	06/19/74	6.20
TROJAN	12/23/75	4.69
TURKEY POINT 3	11/02/72	7.83
TURKEY POINT 4	06/21/73	7.20
VERMONT YANKEE 1	09/20/72	7.95
YANKEE-ROWE 1	11/10/60	19.81
ZION 1	06/28/73	7.18
ZION 2	12/26/73	6.68
TOTAL		465.67 YEARS

PERMANENTLY OR INDEFINITELY SHUTDOWN UNITS

UNIT NAME	FIRST ELEC. GENERATION	SHUTDOWN OR ELECTRICAL CESSATION DATE	YEARS EXPERIENCE
BONUS	08/14/64	06/01/68	3.80
CVTR	12/18/63	01/01/67	3.04
DRESDEN 1	04/15/60	10/31/78	18.54
ELK RIVER	08/24/63	02/01/68	4.44
FERMI 1	08/05/66	11/29/72	6.32
HALLAM	05/29/63	09/01/64	1.26
HUMBOLDT BAY	04/18/63	07/02/76	13.21
INDIAN POINT 1	09/16/62	10/31/74	12.12
PATHFINDER	07/25/66	10/01/67	1.19
PEACH BOTTOM 1	01/27/67	11/01/74	7.76
PIQUA	11/04/63	01/01/66	2.16
THREE MILE ISLAND 2	04/21/78	03/28/79	.93
TOTAL			74.77 YEARS

NON—POWER REACTORS IN THE U.S.

1. RESEARCH REACTORS

State	Location	Licensee	Reactor Type	Docket	License Number	Date Issued	Authorized Power Level (KW)
Alabama	1. Tuskegee	Tuskegee Institute	AGN 201 No. 102	50-406	R-122	08-30-74	.0001
Arizona	2. Tucson	U. of Arizona	TRIGA Mk. I	50-113	R-52	12-05-58	100
California	3. Los Angeles	U. of California	Argonaut	50-142	R-71	10-03-60	100
	4. Berkeley	U. of California	TRIGA Mark III	50-224	R-101	08-10-66	1000
	5. Irvine	U. of California	TRIGA Mark I	50-326	R-116	11-24-69	250
	6. Santa Barbara	U. of California	L-77	50-433	R-124	12-03-74	.01
	7. Canoga Park	Rockwell International Corporation	L-77	50-94	R-40	05-17-58	.01
	8. Hawthorne	Northrop Corporate Laboratories	TRIGA Mark F	50-187	R-90	03-04-63	1000
	9. San Luis Obispo	California State Polytechnic College	AGN 201 No. 100	50-394	R-121	05-16-73	.0001
	10. Alameda County	General Electric Co.	NTR	50-73	R-33	10-31-57	100
	11. San Diego	General Atomic Co.	TRIGA Mark I	50-89	R-38	05-03-58	250
	12. San Diego	General Atomic Co.	TRIGA Mark F	50-163	R-67	07-01-60	1500
	13. San Ramon	Aerotest Operations, Inc.	TRIGA (Industrial)	50-228	R-98	07-02-65	250
Colorado	14. Jefferson County	U.S. Geological Survey. Dept. of Interior	TRIGA Mark I	50-274	R-113	02-24-69	1000
Delaware	15. Newark	U. of Delaware	AGN-201 No. 113	50-98	R-43	07-03-58	.0001
District of Columbia	16. —	Catholic U.	AGN-201 No. 101	50-77	R-31	11-15-67	.0001
Florida	17. Gainesville	U. of Florida	Argonaut	50-83	R-56	05-21-59	100
Georgia	18. Atlanta	Georgia Institute of Technology	AGN-201 No. 104	50-276	R-111	04-19-68	.0001
	19. Atlanta	Georgia Institute of Technology	Heavy Water	50-160	R-97	12-29-64	5000
Idaho	20. Pocatello	Idaho State U.	AGN-201 No. 103	50-284	R-110	10-11-67	.0001
Illinois	21. Urbana	U. of Illinois	Advanced TRIGA	50-151	R-115	07-22-69	1500
	22. Urbana	U. of Illinois	LOPRA	50-356	R-117	12-27-71	10
	23. Zion	Westinghouse Electric Corporation	NTR	50-87	R-119	01-28-72	10
Indiana	24. West Lafayette	Purdue U.	Lockheed	50-182	R-87	08-16-62	10

RESEARCH REACTORS — CONT.

State	Location	Licensee	Reactor Type	Docket	License Number	Date Issued	Authorized Power Level (KW)
Iowa	25. Ames	Iowa State U.	UTR-10	50-116	R-59	10-16-59	10
Kansas	26. Lawrence	U. of Kansas	Lockheed	50-148	R-78	06-23-61	250
	27. Manhattan	Kansas State U.	TRIGA	50-188	R-88	10-16-62	250
Maryland	28. College Park	U. of Maryland	TRIGA	50-166	R-70	10-14-60	250
	29. Bethesda	Armed Forces Radiobiology Research Institute	TRIGA	50-170	R-84	06-26-62	1000
Massachusetts	30. Lowell	U. of Lowell	GE	50-223	R-125	12-24-74	1000
	31. Cambridge	Massachusetts Institute of Technology	Heavy Water Reflected	50-20	R-37	06-09-58	5000
	32. Worcester	Worcester Polytechnic Institute	GE	50-134	R-61	12-16-59	10
Michigan	33. Ann Arbor	U. of Michigan	Pool	50-2	R-28	09-13-57	2000
	34. Lansing	Michigan State U.	TRIGA Mark I	50-294	R-114	03-21-69	250
	35. Midland	Dow Chemical Co.	TRIGA Mark I	50-264	R-108	07-03-67	100
Missouri	36. Rolla	U. of Missouri	Pool	50-123	R-79	11-21-61	200
	37. Columbia	U. of Missouri	Tank	50-186	R-103	10-11-66	10,000
Nebraska	38. Omaha	V.A. Hospital	TRIGA	50-131	R-57	06-26-59	18
New Mexico	39. Albuquerque	U. of New Mexico	AGN-201M No. 112	50-252	R-102	09-17-66	.005
New York	40. Buffalo	State U. of New York	PULSTAR	50-57	R-77	03-24-61	2000
	41. Sterling Forest	Union Carbide Corp.	Pool	50-54	R-81	09-07-61	5000
	42. New York City	Columbia U.	TRIGA Mark II	50-208	R-128	04-14-77	250
	43. Ithaca	Cornell U.	ZPR	50-97	R-89	12-11-62	.1
	44. Ithaca	Cornell U.	TRIGA Mk. II	50-157	R-80	01-11-62	100
	45. Riverdale	Manhattan College	Tank	50-199	R-94	03-24-64	.0001
North Carolina	46. Raleigh	North Carolina State U.	PULSTAR	50-297	R-120	08-25-72	1000
Ohio	47. Columbus	Ohio State U.	Pool	50-150	R-75	02-24-61	10
Oklahoma	48. Norman	U. of Oklahoma	AGN-211 No. 102	50-112	R-53	12-29-58	.015
Oregon	49. Corvallis	Oregon State U.	AGN-201 No. 114	50-106	R-51	11-13-58	.0001
	50. Corvallis	Oregon State U.	TRIGA Mark II	50-243	R-106	03-07-67	1000
	51. Portland	Reed Institute	TRIGA Mk. I	50-288	R-112	07-02-68	250
Pennsylvania	52. University Park	Pennsylvania State U.	TRIGA Mk. III	50-5	R-2	07-08-55	1000

NON–POWER REACTORS IN THE U.S.

RESEARCH REACTORS – CONT.

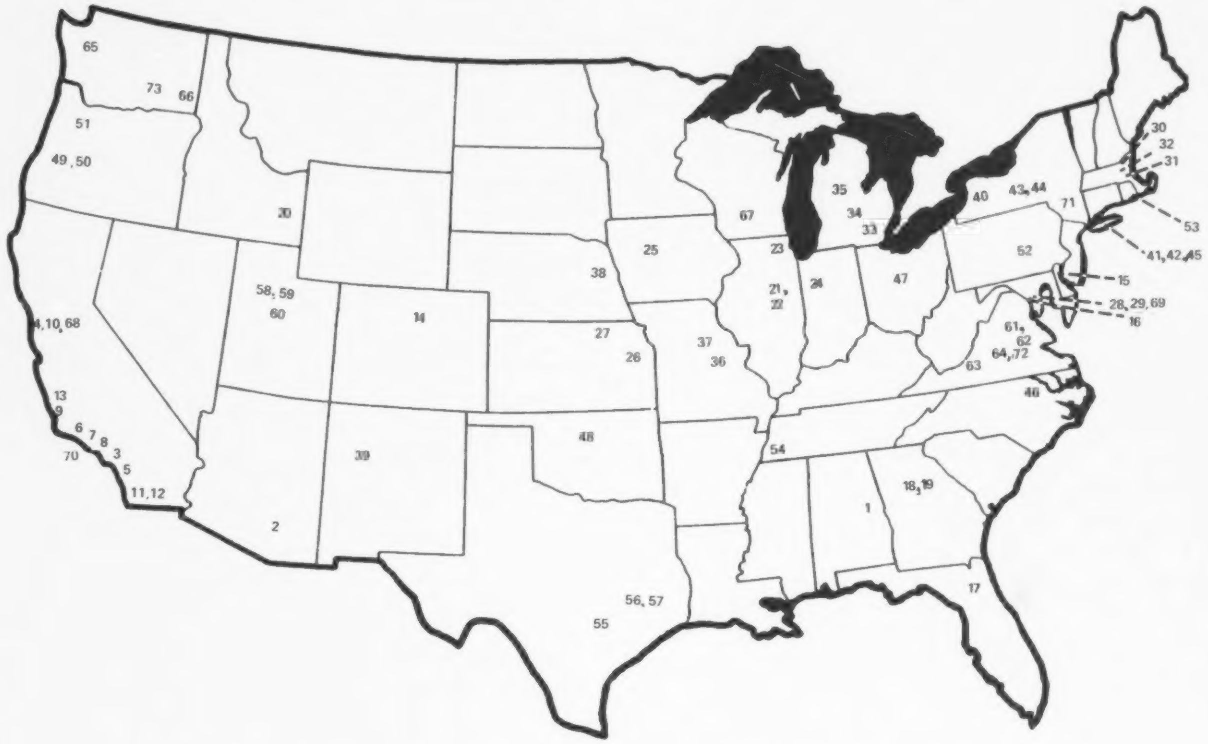
State	Location	Licensee	Reactor Type	Docket	License Number	Date Issued	Authorized Power Level (KW)
Rhode Island	53. Ft. Kearney, Narragansett	Rhode Island & Providence Plantations AEC	GE Pool	50-193	R-95	07-21-64	2000
Tennessee	54. Shelby County	Memphis State U.	AGN-201 No. 108	50-538	R-127	12-10-76	.0001
Texas	55. Austin	U. of Texas	TRIGA Mk. I	50-192	R-92	08-02-63	250
	56. College Station	Texas A&M U.	TRIGA	50-128	R-83	12-07-61	1000
	57. College Station	Texas A&M U.	AGN-201M No. 106	50-59	R-23	08-26-57	.005
Utah	58. Salt Lake City	U. of Utah	AGN-201M No. 107	50-72	R-25	09-12-57	.005
	59. Salt Lake City	U. of Utah	TRIGA Mark I	50-407	R-126	09-30-75	100
	60. Provo	Brigham Young U.	L-77	50-262	R-109	09-07-67	.01
Virgina	61. Charlottesville	U. of Virgina	Pool	50-62	R-66	06-27-60	2000
	62. Charlottesville	U. of Virginia	Cavalier	50-396	R-123	09-24-74	.1
	63. Blacksburg	Virginia Polytechnic Institute	UTR-10	50-124	R-62	12-18-59	100
	64. Lynchburg	Babcock & Wilcox Co.	LPR	50-99	R-47	09-05-58	1000
Washington	65. Seattle	U. of Washington	Argonaut	50-139	R-73	03-31-61	100
	66. Pullman	Washington State U.	TRIGA	50-27	R-76	03-06-61	1000
Wisconsin	67. Madison	U. of Wisconsin	TRIGA	50-156	..-74	11-23-60	1000

2. EXPERIMENTAL AND TEST REACTORS

State	Location	Licensee	Reactor Type	Docket	License Number	Date Issued	Authorized Power Level (MWt)
California	68. Alameda Co.	General Electric Co.	GETR	50-70	TR-1	01-07-59	50
Maryland	69. Gaithersburg	National Bureau of Standards	TEST	50-184	TR-5	11-13-67 POL 06-30-70 OL	10

3. CRITICAL EXPERIMENT FACILITIES

State	Location	Licensee	Docket	License Number	Date Issued	Experiments
California	70. Ventura Co.	Rockwell International Corp. (Possesion Only)	50-147	CX-17	10-03-60	AETR Experiments (Terminal Requested).
New York	71. Schenectady	Rensselaer Polytechnic Institute	50-225	CX-22	07-03-64	
Virginia	72. Lynchburg	Babcock & Wilcox Co.	50-13	CX-10	10-22-58	NMSR Experiments
Washington	73. Richland	Battelle Memorial Institute	50-380	CX-26	11-29-71	Plutonium Recycle Critical Facility



Source: NRC Facilities License Application Record (FLAR) 06-30-77.

ABBREVIATIONS

AGN	Aerojet General Nuclear	These reactors were constructed by Aeroject at San Ramon, California, and then transferred to the current location.
ZPR	Zero Power Reactor	
LPR	Low Power Reactor	

DISTRIBUTION LIST—NRC

INTERNAL DISTRIBUTION

Office of the Commissioners	5
Atomic Safety and Licensing Appeal Panel	3
Advisory Committee on Reactor Safeguards	6
Office of Inspector and Auditor	1
Office of Policy Evaluation	1
Office of the General Counsel	1
Office of Public Affairs	21
Office of Congressional Affairs	3
Office of the Executive Director for Operations	7
Office of Administration	2
Office of the Executive Legal Director	2
Office of the Controller	2
Office for Analysis and Evaluation of Operational Data	2
Office of International Programs	3
Office of State Programs	1
Office of Management and Program Analysis	14
Office of Standards Development	1
Office of Nuclear Material Safety and Safeguards	2
– Division of Fuel Cycle and Material Safety	3
– Division of Safeguards	2
Office of Nuclear Reactor Regulation Division of Licensing Division of Engineering Division of Systems Integration Division of Human Factors Safety Division of Safety Technology	138
Office of Nuclear Regulatory Research	4
Office of Inspection and Enforcement	7
– Region I	11
– Region II	10
– Region III	10
– Region IV	10
– Region V	7
	279

EXTERNAL DISTRIBUTION

SPECIAL REQUESTS

Air Force	1
Congress	10
Department of Energy	32
Government Accounting Office	1
Environmental Protection Agency	1
Electric Power Research Institute	1
Argonne National Laboratory	3
Federal Power Commission	1
Bureau of Mines	1
Department of Agriculture	1
Department of Commerce	1
Department of the Interior	1
	54

OTHER

GPO Depository	440
GPO Stores	75
National Technical Information Service	25
Subscriptions (NTIS)	250
Colleges and Libraries (including Public Document Rooms)	128
Utilities and Other Requests	227
	1145

15/4-10

Nuclear Regulatory Commission

NUREG 0020
VOL. 4 NO. 10
OCTOBER 1980

OPERATING UNITS STATUS REPORT

DATA AS OF 09/30/80

NEWARK PUBLIC LIBRARY
DEC 1 9 1980
U. S. GOVT. PUBS.
DEPOSITORY 376

LICENSED OPERATING REACTORS

Y3.N88:

Available from
GPO Sales Program
Division of Technical Information and Document Control
U.S. Nuclear Regulatory Commission
and
National Technical Information Service
Springfield, Virginia 22161
(703) 326-8543
A Year's Subscription, 12 Issues, for this Publication is
$120.00 North American Continent (240.00 Other).
Multiple Subscriptions Entered at the Same Time to the Same Address
Cost $90.00 North American Continent ($180.00 Other) for each Additional Subscription.

Single copies are $12.50 North American Continent ($25.00 Other).

This document is provided free of charge to those who participate in the data collection program, and to the Nuclear Regulatory Commission's local public document rooms.

Issued by the U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

PUBLISHED: NOVEMBER 12, 1980

NUREG 0020
VOL. 4 NO. 10
OCTOBER 1980

Nuclear Regulatory Commission

OPERATING UNITS STATUS REPORT

DATA AS OF 09/30/80

LICENSED OPERATING REACTORS

AUTHORIZATION AND CLEARANCE

THE U.S. NUCLEAR REGULATORY COMMISSION'S OFFICE OF MANAGEMENT AND PROGRAM ANALYSIS PUBLISHES THIS MONTHLY STATUS REPORT "AS PART OF THE REPORTING REQUIREMENTS IN SECTION 50.36 OF 10 CFR PART 50 UNDER GAO CLEARANCE NUMBER B-180225, WITH AN EXPIRATION DATE OF SEPTEMBER 30, 1981," AS STATED IN THE OCTOBER 3, 1978 LETTER FROM JOHN M. LOVELADY, ASSISTANT DIRECTOR, GENERAL GOVERNMENT DIVISION, U. S. GENERAL ACCOUNTING OFFICE, TO J.M. FELTON, DIRECTOR, DIVISION OF RULES AND RECORDS, U.S. NUCLEAR REGULATORY COMMISSION.

STATEMENT OF PURPOSE

The U. S. Nuclear Regulatory Commission's monthly OPERATING UNITS STATUS REPORT — LICENSED OPERATING REACTORS, provides data on the operation of nuclear units as timely and accurately as possible. This information is collected by the Office of Management and Program Analysis from the Headquarters Staff of NRC's Office of Inspection and Enforcement, from NRC's Regional Offices, and from utilities. Since all of the data concerning operation of the units is provided by the utility operators less than two weeks after the end of the month, necessary corrections to published information are shown on the ERRATA page.

This report is divided into three sections: the first contains monthly highlights and statistics for commercial operating units, and errata from previously reported data; the second is a compilation of detailed information on each unit, provided by NRC Regional Offices, IE Headquarters and the Utilities; and the third section is an appendix for miscellaneous information such as spent fuel storage capability, reactor-years of experience and non-power reactors in the United States.

The percentage computations, Items 19 through 24 in Section 2, the vendor capacity factors on page 1-4, and actual vs. potential energy production on Page 1-2 are computed from actual power production from the period of consideration. The percentages listed in power generation on Page 1-2 are computed as an arithmetical average. The factors for the life-span of each unit (the "Cumulative" column) are reported by the utility and are not entirely re-computed by NRC. Utility power production data is checked for consistency with previously submitted statistics.

In some cases, the numbers in the "Cumulative" column may have been totaled from a date that is different from the date of commercial operation. In these cases, the column is noted with an explanation of the nature of the change on each individual plant page in Section 2.

It is hoped this status report proves informative and helpful to all agencies and individuals interested in analyzing trends in the nuclear industry which might have safety implications, or in maintaining an awareness of the U. S. energy situation as a whole.

TABLE OF CONTENTS

GLOSSARY ii
INDEX TO LICENSED UNITS iii

SECTION 1 – CURRENT DATA SUMMARIES

MONTHLY HIGHLIGHTS OF COMMERCIAL NUCLEAR POWER UNITS 1-2

- Licensed Power Reactors 1-2
- Power Generation 1-2
- Actual vs. Potential Energy Production 1-2
- Outage Data 1-2
- Derated Units 1-2
- Shutdowns - Number and Reasons 1-3
- Unit Availability, Capacity, and Forced Outage Rate 1-3
- Refueling Schedule 1-3

AVERAGE CAPACITY FACTORS BY VENDOR 1-4

AVERAGE DAILY POWER LEVEL FOR ALL COMMERCIAL OPERATING UNITS 1-5

ERRATA 1-7

SECTION 2 – OPERATING POWER REACTORS

ARKANSAS 1 THROUGH ZION 2 PAGES A–2 TO Z–5

- Facility Data
- Inspection and Enforcement Status
- Reports Received from Licensee
- Operating Status
- Average Daily Power Level (MWe)
- Unit Shutdowns/Reductions

SECTION 3 – APPENDIX

STATUS OF SPENT FUEL STORAGE CAPABILITY 3-2
REACTOR-YEARS OF EXPERIENCE 3-3
NON-POWER REACTORS IN THE U.S. 3-4

GLOSSARY

AVERAGE DAILY POWER LEVEL (MWe)	The net electrical energy generated during the day (measured from 0001 to 2400 hours inclusive) in megawatts hours divided by 24 hours.
LICENSED THERMAL POWER (MWt)	The maximum thermal power of the reactor authorized by the NRC, expressed in megawatts.
DATE OF COMMERCIAL OPERATION	Date unit was declared by utility owner to be available for the regular production of electricity; usually related to satisfactory completion of qualification tests as specified in the purchase contract and to accounting policies and practices of utility.
DESIGN ELECTRICAL RATING (DER) (Net MWe)	The nominal net electrical output of the unit specified by the utility and used for the purpose of plant design.
FORCED OUTAGE	An outage required to be initiated no later than the weekend following discovery of an off-normal condition.
FORCED OUTAGE HOURS	The clock hours during the report period that a unit is unavailable due to forced outages.
GROSS ELECTRICAL ENERGY GENERATED (MWH)	Electrical output of the unit during the report period as measured at the output terminals of the turbine generator, in megawatts hours.
GROSS HOURS	The clock hours from the beginning of a specified situation until its end. For outage durations, the clock hours during which the unit is not in power production.
GROSS THERMAL ENERGY GENERATED (MWH)	The thermal energy produced by the unit during the report period as measured or computed by the licensee in megawatt hours.
HOURS GENERATOR ON-LINE	Also, "Unit Service Hours." The total clock hours in the report period during which the unit operated with breakers closed to the station bus. These hours added to the total outage hours experienced by the unit during the report period, shall equal the hours in the report period.
HOURS IN REPORTING PERIOD	For units in power ascension at the end of the period, the gross hours from the beginning of the period or the first electrical production, whichever comes last, to the end of the period. For units in commercial operation at the end of the period, the gross hours from the beginning of the period or of commercial operation, whichever comes last, to the end of the period or decommissioning, whichever comes first.
HOURS REACTOR CRITICAL	The total clock hours in the report period during which the reactor sustained a controlled chain reaction.
MAXIMUM DEPENDABLE CAPACITY (GROSS) (MDC Gross) (Gross MWe)	Dependable main-unit gross capacity, winter or summer, whichever is smaller. The dependable capacity varies because the unit efficiency varies during the year due to cooling water temperature variations. It is the gross electrical output as measured at the output terminals of the turbine generator during the most restrictive seasonal conditions (usually summer).
MAXIMUM DEPENDABLE CAPACITY (NET) (MDC Net) (Net MWe)	Maximum Dependable Capacity (Gross) less the normal station service loads.
NAMEPLATE RATING (Gross MWe)	The nameplate power designation of the generator in megavolt amperes (MVA) times the nameplate power factor of the generator. NOTE: The nameplate rating of the generator may not be indicative of the maximum or dependable capacity, since some other item of equipment of a lesser rating (e.g., turbine) may limit unit output.
NET ELECTRICAL ENERGY GENERATED	Gross electrical output of the unit measured at the output terminals of the turbine generator during the reporting period, minus the normal station service electrical energy utilization. If this quantity is less than zero, a negative number should be recorded.
OUTAGE	A situation in which no electrical production takes place.
OUTAGE DATE	As reported on Appendix D of Reg. Guide 1.16, the date of the start of the outage. If continued from a previous month, report the same outage date but change "Method of Shutting Down Reactor" to "4 (Other)" and add a note: "Continued from previous month."
OUTAGE DURATION	The total clock hours of the outage measured from the beginning of the report period or the outage, whichever comes last, to the end of the report period or the outage, whichever comes first.
OUTAGE NUMBER	A number unique to the outage assigned by the licensee. The same number is reported each month in which the outage is in progress. One format is "76-05" for the fifth outage to occur in 1976.
PERIOD HOURS	See "Hours in Reporting Period."
POWER REDUCTION	A reduction in the Average Daily Power Level of more than 20% from the previous day. All power reductions are defined as outage of zero hours durations for the purpose of computing unit service and availability factors, and forced outage rate.

REACTOR AVAILABLE HOURS	The total clock hours in the report period during which the reactor was critical or was capable of being made critical. (Reactor Reserve Shutdown Hours + Hours Reactor Critical.)
REACTOR AVAILABILITY FACTOR	$\frac{\text{Reactor Available Hours} \times 100}{\text{Period Hours}}$
REACTOR RESERVE SHUTDOWN	The cessation of criticality in the reactor for administrative or other similar reasons when operation could have been continued.
REACTOR RESERVE SHUTDOWN HOURS	The total clock hours in the report period during which the reactor is in reserve shutdown mode. NOTE: No credit is given for NRC imposed shutdowns.
REACTOR SERVICE FACTOR	$\frac{\text{Hours Reactor Critical} \times 100}{\text{Period Hours}}$
REPORT PERIOD	Usually, the preceding calendar month. Can also be the preceding calendar year, (Year-to-Date), or the life-span of a unit (cumulative).
RESTRICTED POWER LEVEL	Maximum net electrical generation to which the unit is restricted during the report period due to the state of equipment, external conditions, administrative reasons, or a direction by NRC.
SCHEDULED OUTAGE	Planned removal of a unit from service for refueling, inspection, training, or maintenance. Those outages which do not fit the definition of "Forced Outage" perforce are "Scheduled Outages."
STARTUP AND POWER ASCENSION TEST PHASE	Period following initial criticality during which the unit is tested at successively higher levels, culminating with operation at full power for a sustained period and completion of warranty runs. Following this phase, the utility generally considers the unit to be available for commercial operation.
UNIT	The set of equipment uniquely associated with the reactor, including turbine generators, and ancillary equipment, considered as a single electrical energy production facility.
UNIT AVAILABLE HOURS	The total clock hours in the report period during which the unit operated on-line or was capable of such operation. (Unit Reserve Shutdown Hours + Hours Generator On-Line.)
UNIT AVAILABILITY FACTOR*	$\frac{\text{Unit Available Hours} \times 100}{\text{Period Hours}}$
UNIT CAPACITY FACTORS	
– Using Licensed Thermal Power	$\frac{\text{Gross Thermal Energy Generated} \times 100}{\text{Period Hours} \times \text{Lic. Thermal Power}}$
– Using Nameplate Rating	$\frac{\text{Gross Electrical Energy Generated} \times 100}{\text{Period Hours} \times \text{Nameplate Rating}}$
– Using DER	$\frac{\text{Net Electrical Energy Generated} \times 100}{\text{Period Hours} \times \text{DER}}$
– Using MDC Gross	$\frac{\text{Gross Electrical Energy Generated} \times 100}{\text{Period Hours} \times \text{MDC Gross}}$
– Using MDC Net	$\frac{\text{Net Electrical Energy Generated} \times 100}{\text{Period Hours} \times \text{MDC Net}}$

NOTE: If MDC Gross and/or MDC Net have not been determined, the DER is substituted for this quantity for Unit Capacity Factor calculations.

UNIT FORCED OUTAGE RATE	$\frac{\text{Forced Outage Hours}}{\text{Unit Service Hours} + \text{Forced Outage Hours}}$
UNIT RESERVE SHUTDOWN	The removal of the unit from on-line operation for economic or other similar reasons when operation could have been continued.
UNIT RESERVE SHUTDOWN HOURS	The total clock hours in the report period during which the unit was in reserve shutdown mode.
UNIT SERVICE FACTOR	$\frac{\text{Unit Service Hours} \times 100}{\text{Period Hours}}$
UNIT SERVICE HOURS	See "Hours Generator On-Line."

INDEX TO LICENSED UNITS

ARKANSAS 1	A-3
ARKANSAS 2	A-5
BEAVER VALLEY 1	B-3
BIG ROCK POINT 1	B-5
BROWNS FERRY 1	B-7
BROWNS FERRY 2	B-9
BROWNS FERRY 3	B-11
BRUNSWICK 1	B-13
BRUNSWICK 2	B-15
CALVERT CLIFFS 1	C-3
CALVERT CLIFFS 2	C-5
COOK 1	C-7
COOK 2	C-9
COOPER STATION	C-11
CRYSTAL RIVER 3	C-13
DAVIS-BESSE 1	D-3
DRESDEN 1	D-5
DRESDEN 2	D-7
DRESDEN 3	D-9
DUANE ARNOLD	D-11
FARLEY 1	F-3
FITZPATRICK	F-5
FORT CALHOUN 1	F-7
FORT ST VRAIN	F-9
GINNA	G-3
HADDAM NECK	H-3
HATCH 1	H-5
HATCH 2	H-7
HUMBOLDT BAY	H-9
INDIAN POINT 2	I-3
INDIAN POINT 3	I-5
KEWAUNEE	K-3
LA CROSSE	L-3
MAINE YANKEE	M-3
MILLSTONE 1	M-5
MILLSTONE 2	M-7
MONTICELLO	M-9
NINE MILE POINT 1	N-3
NORTH ANNA 1	N-5
NORTH ANNA 2	N-7
OCONEE 1	O-3
OCONEE 2	O-5
OCONEE 3	O-7
OYSTER CREEK 1	O-9
PALISADES	P-3
PEACH BOTTOM 2	P-5
PEACH BOTTOM 3	P-7
PILGRIM 1	P-9
POINT BEACH 1	P-11
POINT BEACH 2	P-13
PRAIRIE ISLAND 1	P-15
PRAIRIE ISLAND 2	P-17
QUAD CITIES 1	Q-3
QUAD CITIES 2	Q-5
RANCHO SECO 1	R-3
ROBINSON 2	R-5
SALEM 1	S-3
SAN ONOFRE 1	S-5
SEQUOYAH 1	S-7
ST LUCIE 1	S-9
SURRY 1	S-11
SURRY 2	S-13
THREE MILE ISLAND 1	T-3
THREE MILE ISLAND 2	T-5
TROJAN	T-7
TURKEY POINT 3	T-9
TURKEY POINT 4	T-11
VERMONT YANKEE 1	V-3
YANKEE-ROWE 1	Y-3
ZION 1	Z-3
ZION 2	Z-5

SECTION 1 — CURRENT DATA SUMMARIES

MONTHLY HIGHLIGHTS OF COMMERCIAL NUCLEAR POWER UNITS

FOR MONTH OF

SEPTEMBER 1980

LICENSED POWER REACTORS:

NUMBER		CAPACITY[1] MWe (Net)
66	IN COMMERCIAL OPERATION	48,663
2	IN POWER ASCENSION PHASE	2,046
68[2]	LICENSED TO OPERATE	50,709
1[3]	LICENSED FOR FUEL LOADING AND LOW POWER TESTING	

POWER GENERATION	REPORT MONTH	PREVIOUS MONTH	YEAR-TO-DATE
1. GROSS ELECTRICAL (MWhe)	24,544,105	25,581,521	191,450,015
2. NET ELECTRICAL (MWhe)	23,356,369	24,312,485	182,174,974
3. AVG. UNIT SERVICE FACTOR (%)	74.1	77.8	64.4
4. AVERAGE UNIT AVAILABILITY FACTOR (%)	74.1	77.8	64.5
5. AVERAGE UNIT CAPACITY FACTOR (MDC) (%)	65.5	65.8	56.7
6. AVG. UNIT CAPACITY FACTOR (DER) (%)	63.5	64.0	55.2
7. FORCED OUTAGE RATE (%)	11.0	11.2	11.6

NOTES

[1] BASED UPON MAXIMUM DEPENDABLE CAPACITY; DESIGN ELECTRIC RATING USED IF MDC NOT DETERMINED.

[2] EXCLUDES DRESDEN 1 (DER = 200), HUMBOLDT BAY (DER = 65), AND THREE MILE ISLAND 2 (DER = 906) WHICH HAVE OPERATING LICENSES BUT ARE SHUTDOWN INDEFINITELY.

[3] SALEM 2 (DER = 1115) WAS LICENSED ON APRIL 18, 1980

ACTUAL VS. POTENTIAL ENERGY PRODUCTION

			% OF POTENTIAL PRODUCTION
POTENTIAL ENERGY PRODUCTION IN THIS PERIOD BY UNITS IN COMMERCIAL OPERATION (Using Maximum Dependable Capacity Net)		35,037,360 MWHe	
ENERGY ACTUALLY PRODUCED DURING THIS REPORT PERIOD	=	23,356,369 MWHe NET	66.7
ENERGY NOT PRODUCED DURING THIS REPORT PERIOD DUE TO SCHEDULED OUTAGES (NET)	=	5,314,126 MWHe	15.2
ENERGY NOT PRODUCED DURING THIS REPORT PERIOD DUE TO FORCED OUTAGES (NET)	=	3,314,902 MWHe	9.5
ENERGY NOT PRODUCED DURING THIS REPORT PERIOD FOR OTHER REASONS (NET)	=	3,051,962 MWHe	8.7
TOTAL		35,037,359 MWHe =	100%
ENERGY NOT PRODUCED THIS REPORT PERIOD DUE TO NRC IMPOSED SHUTDOWNS AND RESTRICTIONS	=	1,171,334 MWHe	3.3
NUMBER OF UNITS WITH POWER RESTRICTIONS IMPOSED BY NRC	=	2	

OUTAGE DATA

	NUMBER	HOURS	PERCENT OF CLOCK TIME	MWHE LOST PRODUCTION
1. FORCED OUTAGES DURING MONTH	60	5,097.4	10.7	3,314,902
2. SCHEDULED OUTAGES DURING THE MONTH	19	7,198.2	15.1	5,314,126
3. TOTAL	79	12,295.6	25.9	8,629,029

MWHE LOST PRODUCTION = Down time x maximum dependable capacity net

DERATED UNITS

UNIT NAME	MAXIMUM DEPENDABLE CAPACITY (MWe - NET)	POWER LIMIT (MWe - NET)
COOPER STATION	764	640
†FORT ST VRAIN	330	231
INDIAN POINT 2	849	824
NINE MILE POINT 1	610	598
OYSTER CREEK 1	620	550
†PALISADES	635	580
POINT BEACH 1	495	390
ROBINSON 2	2300 MWT	2200 MWT

†Restricted by NRC

MONTHLY HIGHLIGHTS OF COMMERCIAL NUCLEAR POWER UNITS

FOR MONTH OF

SEPTEMBER 1980

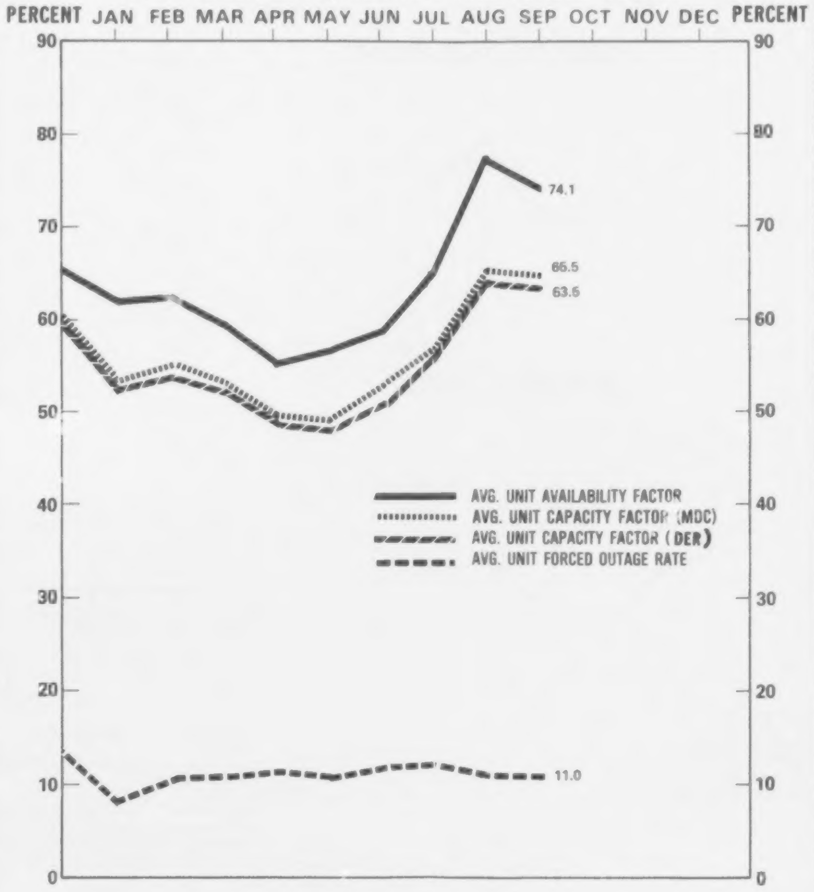
REASONS FOR SHUTDOWN

REASON	NUMBER	TOTAL HOURS LOST
A - Equipment Failure	43	3,012.4
B - Maintenance or Test	12	3,022.7
C - Refueling	8	4,562.6
D - NRC Restriction	2	1,440.0
E - Operator Training and License Examination	0	0
F - Administrative	2	85.8
G - Operational Error	8	72.9
H - Other	4	99.2
TOTAL	79	12,295.6

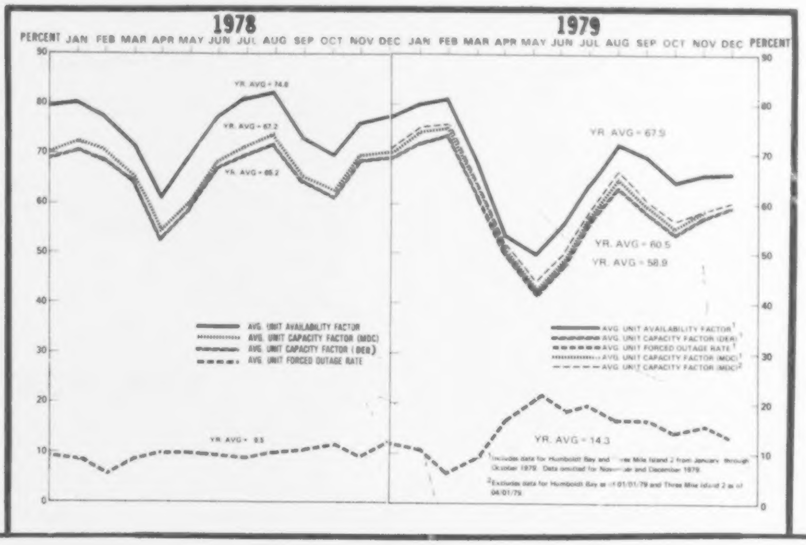
SHUTDOWNS GREATER THAN 72 HOURS EACH

UNIT NAME	REASON	UNIT NAME	REASON
ARKANSAS 1	A	OCONEE 1	A
ARKANSAS 2	B	OCONEE 3	A
BEAVER VALLEY 1	D	OYSTER CREEK 1	B
BROWNS FERRY 2	C	PRAIRIE ISLAND 1	C
BRUNSWICK 2	A,B	PRAIRIE ISLAND 2	B
DAVIS-BESSE 1	C	QUAD CITIES 1	C
DRESDEN 2	A	ROBINSON 2	C
FARLEY 1	F	SALEM 1	C
FORT ST VRAIN	B	SAN ONOFRE 1	B
KEWAUNEE	A	SURRY 1	A
LA CROSSE	A	THREE MILE ISLAND 1	D
MAINE YANKEE	A	VERMONT YANKEE 1	C
MILLSTONE 2	C	YANKEE-ROWE 1	A

UNIT AVAILABILITY, CAPACITY AND FORCED OUTAGE



UNIT AVAILABILITY, CAPACITY AND FORCED OUTAGE

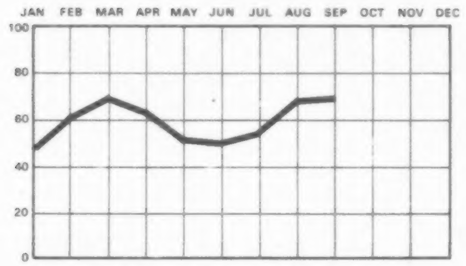


REFUELING SCHEDULE

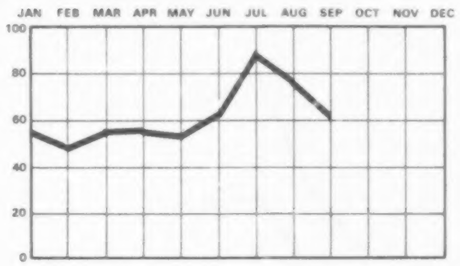
Please refer to the table on page 3-2 for the scheduled refueling date for each nuclear powerplant in this publication.

VENDOR AVERAGE CAPACITY FACTORS

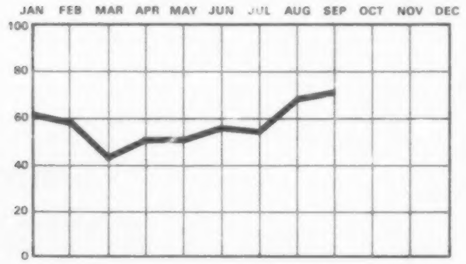
WESTINGHOUSE



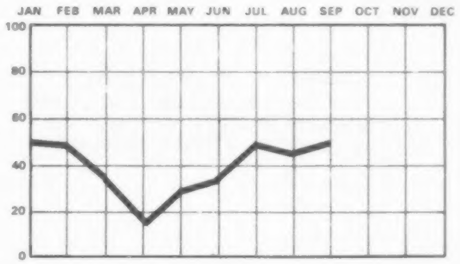
COMBUSTION ENGINEERING



GENERAL ELECTRIC



BABCOCK & WILCOX



NOTE: THIS DISPLAY OF AVERAGE CAPACITY FACTORS PROVIDES A GENERAL PERFORMANCE COMPARISON OF PLANTS SUPPLIED BY THE FOUR NUCLEAR STEAM SUPPLY SYSTEM VENDORS. ONE MUST BE CAREFUL WHEN DRAWING CONCLUSIONS REGARDING THE REASONS FOR THE PERFORMANCE LEVELS INDICATED, SINCE PLANT PERFORMANCE MAY BE AFFECTED BY UNSPECIFIED FACTORS SUCH AS: (1) VARIOUS PLANT DESIGNS AND MODELS ARE INCLUDED FOR EACH VENDOR; (2) TURBINE/GENERATORS AND (3) DIFFERENT ARCHITECT/ENGINEERS ARE ALSO INVOLVED.

SEPTEMBER 1980

OPERATIONS

GE BWR's	CFMDC
BROWNS FERRY 1	88.4
BROWNS FERRY 2	14.1
BROWNS FERRY 3	79.2
BRUNSWICK 1	75.7
BRUNSWICK 2	7.0
COOPER STATION	76.9
DRESDEN 2	46.6
DRESDEN 3	95.4
DUANE ARNOLD	85.3
FITZPATRICK	94.1
HATCH 1	83.6
HATCH 2	82.5
MILLSTONE 1	33.1
MONTICELLO	97.0
NINE MILE POINT 1	85.3
OYSTER CREEK 1	71.7
PEACH BOTTOM 2	97.1
PEACH BOTTOM 3	97.8
PILGRIM 1	93.4
QUAD CITIES 1	.0
QUAD CITIES 2	93.0
VERMONT YANKEE 1	73.5

22 UNITS

W PWR's	
BEAVER VALLEY 1	.0
COOK 1	89.5
COOK 2	95.0
FARLEY 1	69.5
GINNA	98.8
HADDAM NECK	88.8
INDIAN POINT 2	84.1
INDIAN POINT 3	74.3
KEWAUNEE	79.2
NORTH ANNA 1	93.2
POINT BEACH 1	74.7
POINT BEACH 2	97.2
PRAIRIE ISLAND 1	.0
PRAIRIE ISLAND 2	67.5
ROBINSON 2	.0
SALEM 1	27.5
SAN ONOFRE 1	.0
SUPRY 1	28.8
SURRY 2	88.7
TROJAN	98.9
TURKEY POINT 3	99.9
TURKEY POINT 4	98.5
YANKEE-ROWE 1	.0
ZION 1	86.7
ZION 2	77.2

25 UNITS

CE PWR's	CFMDC
ARKANSAS 2	10.1
CALVERT CLIFFS 1	94.2
CALVERT CLIFFS 2	77.2
FORT CALHOUN 1	87.9
MAINE YANKEE	63.9
MILLSTONE 2	.0
PALISADES	82.8
ST LUCIE 1	99.0

8 UNITS

B&W PWR's	CFMDC
ARKANSAS 1	19.1
CRYSTAL RIVER 3	74.0
DAVIS-BESSE 1	.0
OCONEF 1	65.9
OCONEE 2	87.4
OCONEE 3	79.2
RANCHO SECO 1	86.0
THREE MILE ISLAND 1	.0

8 UNITS

	GE BWR's	W PWR's	CE PWR's	B&W PWR's	All PWR's
NET ELECTRICAL PRODUCTION	9,082,875	9,040,517	2,662,139	2,511,850	14,214,506
MDC NET	17,154	18,292	6,036	6,737	31,065
CFMDC	73.5	68.6	61.3	51.8	63.6

Capacity factor in this page, denoted as CFMDC, is a function of the net maximum dependable capacity. See the corresponding definition in the Glossary. The vendor averages are computed by the formula:

$$\frac{\text{Net Electrical Energy Produced by Vendor}}{\text{Potential Electrical Production By Vendor In This Month.}} \text{ X } 100\%$$

Units Excluded	Reason for Exclusion
Dresden 1 Humboldt Bay Three Mile Island 2	• Unit Shutdown Indefinitely
Big Rock Point	Not Representative of Typical BWRs
LaCrosse	Not Representative of Typical BWRs and Atypical Vendor
Fort St. Vrain	Atypical Vendor and Reactor Design

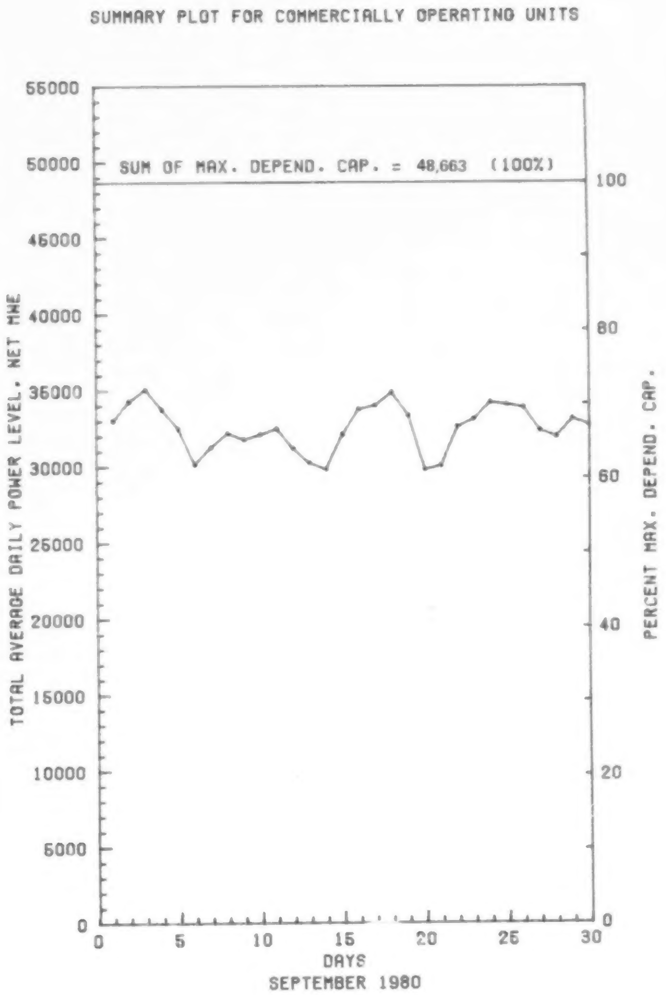
AVERAGE DAILY POWER LEVEL FOR ALL COMMERCIALLY OPERATING UNITS

This chart depicts the average daily power level for the units in commercial operation during the month.

The straight line on the graph labelled "SUM OF MAX. DEPEND. CAP." is plotted at the value shown by summing the separate maximum dependable capacities of the commercially operating units (in Net MWe). The plot shown below the line is calculated by summing the separate average daily power levels of the commercially operating units for each day of the month.

The scale on the left vertical axis runs in 1,000 MWe increments from 0 to 55,000 MWe (Net). The right vertical axis shows the percentage in 10% increments, up to 100% of the "SUM OF MAX. DEPEND. CAP."

It should be recognized that the 100% line would be obtainable only if all of the commercially operating units operated at 100% capacity, 24 hours per day, for the entire month. In other words, since any power generator must occasionally shut down to refuel and/or perform needed maintenance, and also since 100% capacity production is not always required by power demands, the 100% line is a theoretical goal and not a practical one.



blank page

ERRATA

Corrections and Amendments to Previously-Reported Data

(NOTE: These changes are reflected in the data contained in the current report)

CORRECTIONS TO NUREG-0020 ______ ______ *

Page	Unit Name	Item	Correction
	NONE		

MULTIPLE - ISSUE CORRECTIONS

OYSTER CREEK:

AUGUST 1980. ITEM NO. 17 SHOULD BE 317,890

AUGUST 1980. ITEM NO. 18 SHOULD BE 315,837.

SECTION 2 – OPERATING POWER REACTORS

best copy available

ARKANSAS 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Arkansas/Pope 6 miles NW of Russellville
2.	Electric Reliability Council	Southwest Power Pool
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2568
5.	Design Electrical Rating (MWE Net)	850
6.	Date of Initial Criticality	August 6, 1974
7.	Date of Commercial Operation	December 19, 1974
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Dardenelle Reservoir

Utility & Contractor Information

10.	Licensee	Arkansas Power and Light Company
11.	Corporate Address	Ninth & Louisiana Streets Little Rock, Arkansas 72203
12.	Corporate Contact	W. Cavanaugh III Vice Pres., Generation & Construction
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	B&W
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	IV
17.	IE Branch Chief	G. L. Madsen
18.	IE Section Chief	D. M. Hunnicutt
19.	IE Resident Inspector	W. D. Johnson (501) 968-3290
20.	Licensing Project Manager	G. S. Vissing
21.	Docket Number	50-313
22.	License Number/Date of Issue	DPR-51, 5/21/74
23.	Public Document Room	Arkansas Polytechnic College Russellville, Arkansas 72801

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO TECH SPEC 6.8.1, THE LICENSEE FAILED TO FOLLOW PROCEDURE 1602.27 IN THAT ANTI-C CLOTHING WAS STORED FOR USE THAT EXCEEDED FIXED CONTAMINATION LIMITS AND ANTI-C CLOTHING WAS NOT SEGREGATED BY CONTAMINATION LEVEL.
(8015 INFRACTION)

CONTRARY TO 10 CFR 50 APPENDIX B, CRITERION V: (1) THE PLANT SAFETY COMMITTEE WAS NOT ROUTINELY REVIEWING THE AUDIT REPORTS AND AUDIT FINDING REPORT ISSUED BY THE QA ORGANIZATION AS REQUIRED BY SECT 6.3.12 OF PROCEDURE NO. 1000.02: (2) A SINGLE AUDIT FINDING REPORT (AFR'S) WAS ISSUED TO DOCUMENT THE SIX DEFICIENCIES IDENTIFIED IN AUDIT NO. E-80-4 (CONDUCTED IN MAR 1980) AS OPPOSED TO AN INDIVIDUAL AFR FOR EACH DEFICIENCY WHICH WAS REQUIRED BY PARAGRAPH 5.1 OF QA PROCEDURE ANO-14.
(8016 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

Unit was in cold shutdown during much of September to repair a tube leak in "A" OTSG and to clean service water system of blockage by Asian clams and silt.

FACILITY ITEMS (PLANS AND PROCEDURES):

MANAGERIAL ITEMS

PLANT STATUS:

Plant was shut down on 9/5/80 due to a leak in 'A' OTSG.
Plant is expected to return to full power operation on 9/28/80.

LAST IE SITE INSPECTION DATE: July 22 - August 21, 1980

INSPECTION REPORT NO: 50-313/80-15

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
	NONE		

INSPECTION SUMMARY

Inspection conducted during the period of July 21-25, 1980 (80-13): Routine, unannounced inspection of Security Plan; Security Program Audit; Protected Area Physical Barriers; Vital Area Physical Barriers; Lighting; Alarm Stations; Detection Aids; Testing and Maintenance; Power Supply. Within the nine areas inspected, four items of noncompliance were identified (infraction - vital area physical barrier; two infractions involving detection aids; and infraction - intrusion alarm testing).

Management meeting conducted on August 20, 1980 (80-14): Special announced management meeting to discuss the region's first annual evaluation of the licensee's performance relative to activities at the ANO site during the period of January 1, 1979 through August 19, 1980. Of the seventeen functional areas evaluated, the licensee's performance in thirteen areas was such that no change in the frequency or scope of inspection program for those areas is recommended. The licensee's performance in four areas (training, security, reporting, and quality control) was such that increased inspection effort is planned in these areas.

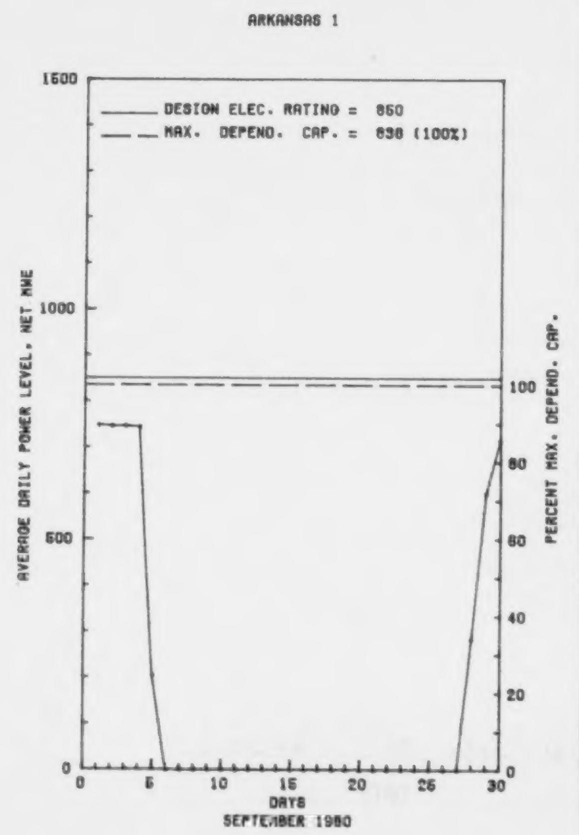
Routine, announced inspection including Surveillance Observation, Inspection conducted during period July 22-August 21, 1980 (80-15): Maintenance, Operational Safety Verification, Followup on IE Bulletins, and Followup on previously identified items.
Within the five (5) areas inspected, one item of noncompliance was identified (infraction - anti-contamination clothing, in one area.

REVIEWED BY OMPA LOEB: **DATE:** 11/4/80

REVIEWED BY NRR DL: **DATE:** 11/6/80

ARKANSAS 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-313
2. Utility Contact: L. S. Bramlett (501) 968-2519
3. Licensed Thermal Power (MWt): 2568
4. Nameplate Rating (Gross MWe): 1003 x 0.9 = 903
5. Design Electrical Rating (Net MWe): 850
6. Maximum Dependable Capacity (Gross MWe): 883
7. Maximum Dependable Capacity (Net MWe): 836
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	6,575.0	50,706.0
12. Hours Reactor Was Critical	179.9	3,457.2	33,595.4
13. Reactor Reserve Shutdown Hours	73.7	647.6	4,895.0
14. Hours Generator On-Line	170.4	3,391.8	32,862.3
15. Unit Reserve Shutdown Hours	0	20.8	817.5
16. Gross Thermal Energy (MWH)	394,986	8,090,744	79,352,788
17. Gross Electrical Energy (MWH)	121,130	2,493,620	26,214,776
18. Net Electrical Energy (MWH)	114,907	2,371,967	25,006,003
19. Unit Service Factor	23.7	51.6	64.8
20. Unit Availability Factor	23.7	51.9	66.4
21. Unit Capacity Factor (MDC Net)	19.1	43.2	59.0
22. Unit Capacity Factor (DER Net)	18.8	42.4	58.0
23. Unit Forced Outage Rate	76.3	35.4	18.2

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
JANUARY 1, 1981 TO MARCH 1, 1981 - REFUELING.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-09	09/05/80	F	549.6	A	1	80-034	CC	HTEXCH	THE OUTAGE WAS CAUSED WHEN A TUBE LEAK DEVELOPED IN THE "A" OTSG. ONE TUBE WAS LEAKING AND WAS PLUGGED. EDDY CURRENT TESTING WAS USED TO CHECK OTHER TUBES FOR INTEGRITY.

SUMMARY

ARKANSAS 1 WAS OFF-LINE FROM THE 5TH THRU THE 27TH DUE TO A TUBE LEAK IN THE "A" OTSG.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

ARKANSAS 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Arkansas/Pope 6 miles NW of Russellville
2.	Electric Reliability Council	Southwest Power Pool
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2815
5.	Design Electrical Rating (MWE Net)	912
6.	Date of Initial Criticality	12-5-78
7.	Date of Commercial Operation	3/26/80
8.	Condenser Cooling Method	Recirculating
9.	Condenser Cooling Water Source	Cooling Tower

Utility & Contractor Information

10.	Licensee	Arkansas Power & Light Company
11.	Corporate Address	Ninth & Louisiana Streets Little Rock, Arkansas 72203
12.	Corporate Contact	W. Cavanaugh III Vice Pres., Generation & Construction
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Combustion Engineering, Inc.
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	IV
17.	IE Branch Chief	G. L. Madsen
18.	IE Section Chief	D. M. Hunnicutt
19.	IE Resident Inspector	W. D. Johnson (501) 968-3290
20.	Licensing Project Manager	R. E. Martin 492-8087
21.	Docket Number	50-368
22.	License Number/Date of Issue	NFP-6, 9/1/78
23.	Public Document Room	Arkansas Polytechnic College Russellville, Arkansas 72801

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO TECH SPEC 6.8.1, THE LICENSEE FAILED TO FOLLOW PROCEDURE 1602.27 IN THAT ANTI-C CLOTHING WAS STORED FOR USE THAT EXCEEDED FIXED CONTAMINATION LIMITS AND ANTI-C CLOTHING WAS NOT SEGREGATED BY CONTAMINATION LEVEL.
(8015 INFRACTION)

CONTRARY TO 10 CFR 50, APPENDIX B, CRITERION V: (1) THE PLANT SAFETY COMMITTEE WAS NOT ROUTINELY REVIEWING THE AUDIT REPORTS AND AUDIT FINDING REPORT ISSUED BY THE QA ORGANIZATION AS REQUIRED BY SECT. 6.3.12 OF PROCEDURE NO. 1000.02: (2) A SINGLE AUDIT FINDING REPORT (AFR'S) WAS ISSUED TO DOCUMENT THE SIX DEFICIENCIES IDENTIFIED IN AUDIT NO. E-80-4 (CONDUCTED IN MAR 1980) AS OPPOSED TO AN INDIVIDUAL AFR FOR EACH DEFICIENCY WHICH WAS REQUIRED BY PARAGRAPH 5.1 OF QA PROCEDURE ANO-14.
(8016 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

Plant was in cold shutdown during month of September due to blockage of service water coolers by Asian clams and silt.

FACILITY ITEMS (PLANS AND PROCEDURES):

MANAGERIAL ITEMS:

PLANT STATUS:

The unit was shut down on 9/4/80 to resolve problem with inadequate service water flow to containment cooling units. Plant is expected to return to full power on 9/30/80.

LAST IE INSPECTION DATE: July 22 - August 21, 1980

INSPECTION REPORT NO: 50-368/80-15

INSPECTION SUMMARY

Inspection conducted during the period of July 21 - 25, 1980 (80-13): Routine, unannounced inspection of Security Plan; Security Program Audit; Protected Area Physical Barriers; Vital Area Physical Barriers; Lighting; Alarm Stations; Detection Aids; Testing and Maintenance; Power Supply. Within the nine areas inspected, four items of non-compliance were identified (infraction - vital area physical barrier; two infractions involving detection aids; and infraction - intrusion alarm testing).

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-57 03L-0	8/5/80	9/3/80	Control element assemblies calculators out of service
80-59 03L-0	7/27/80	8/26/80	COLSS plant computer program failure
80-60 03L-0	7/30/80	8/28/80	Core protection calculator channel failure
80-61 03L-0	7/28/80	8/27/80	Charging pipe suction pipe leak
80-62 03L-0	8/11/80	9/9/80	CPC power uncertainty bias terms less conservative than TS Table 4.3-1
80-63 03L-0	8/22/80	9/18/80	Condensate storage tank level low
80-64 03L-0	8/10/80	9/8/80	Plant protective system channel failures
80-65 03L-0	8/12/80	9/9/80	Containment sump recirculation valve failure
80-66 03L-0	8/11/80	9/9/80	Plant protective system trip
80-67 03L-0	8/23/80	9/16/80	Iodine 131 dose equivalent value high
80-68 03L-0	8/23/80	9/16/80	Charging pump discharge header leak
80-72 01T-0	9/3/80	9/17/80	Service water flow to containment building coolers

Management meeting conducted on August 20, 1980 (80-14): Special announced management meeting to discuss the region's first annual evaluation of the licensee's performance relative to activities at the ANO site during the period of January 1, 1979 through August 19, 1980. Of the seventeen functional areas evaluated, the licensee's performance in thirteen areas was such that no change in the frequency or scope of the inspection program for those areas is recommended. The licensee's performance in four areas (training, security, reporting, and quality control) was such that increased inspection effort is planned in those areas.

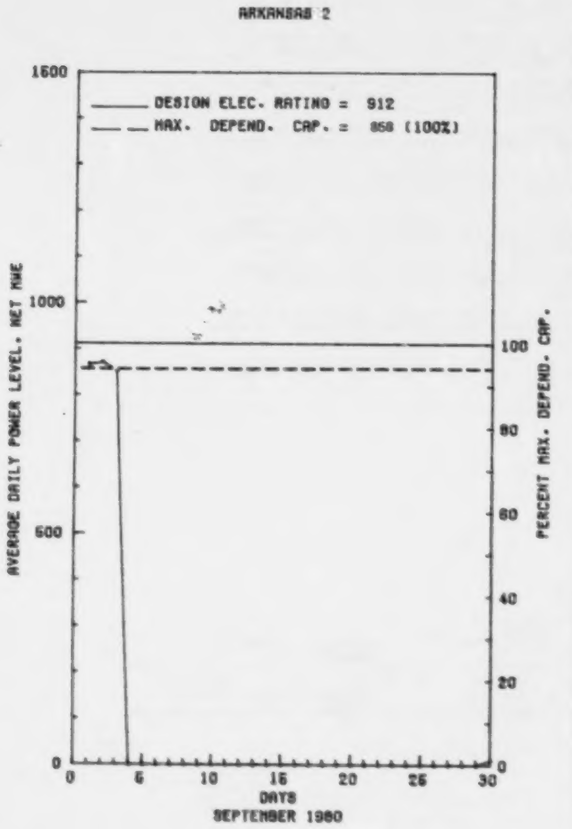
Inspection conducted during period of July 22 - August 21, 1980 (80-15): Routine, announced inspection including Operational Safety Verification, Surveillance Observation, Followup on IE Bulletins, and Followup on previously identified items. Within the four areas inspected, one item of noncompliance was identified (infraction - anti-contamination clothing, in one area.

REVIEWED BY OMPA LOEB: MR Beebe DATE: 11/04/80

REVIEWED BY NRR DL: R Martin DATE: 11/5/80

ARKANSAS 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-368
2. Utility Contact: Rex Pendergraft (501) 968-2519
3. Licensed Thermal Power (MWt): 2815
4. Nameplate Rating (Gross MWe): 959
5. Design Electrical Rating (Net MWe): 912
6. Maximum Dependable Capacity (Gross MWe): 897
7. Maximum Dependable Capacity (Net MWe): 858
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	4,535.0	4,535.0
12. Hours Reactor Was Critical	90.3	3,238.5	3,238.5
13. Reactor Reserve Shutdown Hours	629.7	754.5	734.5
14. Hours Generator On-Line	77.6	3,177.5	3,177.5
15. Unit Reserve Shutdown Hours	.0	75.0	75.0
16. Gross Thermal Energy (MWH)	204,335	7,977,980	7,977,980
17. Gross Electrical Energy (MWH)	65,550	2,589,350	2,589,350
18. Net Electrical Energy (MWH)	62,488	2,467,367	2,467,367
19. Unit Service Factor	10.8	70.1	70.1
20. Unit Availability Factor	10.8	71.7	71.7
21. Unit Capacity Factor (MDC Net)	10.1	63.4	63.4
22. Unit Capacity Factor (DER Net)	9.5	59.7	59.7
23. Unit Forced Outage Rate	89.2	29.9	29.9

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): MARCH 15, 1981 TO MAY 15, 1981 - REFUELING.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-20	09/03/80	F	642.4	B	1	80-072	WA	HTEXCH	OUTAGE OCCURRED DUE TO INADEQUATE SERVICE WATER FLOW THROUGH THE RX BLDG. COOLING COILS. THE RX BLDG. COOLERS WERE CLEANED AND RETURNED TO SERVICE. DURING THE OUTAGE OTHER COOLERS USING SERVICE WATER WERE CHECKED AND/OR CLEANED INSURING PROPER (DESIGN) OPERATION.

SUMMARY

ARKANSAS 2 WAS OFF-LINE FROM THE 3RD THRU THE 29TH FOR CLEANING OF THE REACTOR BUILDING COOLING COILS.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

NRC POWER RESTRICTIONS: NONE

BEAVER VALLEY 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Pennsylvania/Beaver 5 miles E of E. Liverpool, Ohio
2.	Electric Reliability Council	East Central Area Reliability Coordination Agreement
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2660
5.	Design Electrical Rating (MWE Net)	852
6.	Date of Initial Criticality	May 10, 1976
7.	Date of Commercial Operation	10/01/76 (50%) 04/30/77 (100%)
8.	Condenser Cooling Method	Cooling tower/Closed loop
9.	Condenser Cooling Water Source	Ohio River

Utility & Contractor Information

10.	Licensee	Duquesne Light Company
11.	Corporate Address	435 Sixth Avenue Pittsburgh, Pennsylvania 15219
12.	Corporate Contact	C. N. Dunn, Vice President
13.	Architect/Engineer	Stone & Webster
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Section Chief	E. C. MCCABE
19.	IE Resident Inspector	D. A. BECKMAN (412-643-9200/9201)
20.	Licensing Project Manager	W. J. Ross
21.	Docket Number	50-334
22.	License Number/Date of Issue	DPR-66, 1/30/76
23.	Public Document Room	B. F. Jones Memorial Library 633 Franklin Avenue Aliquippa, PA 15001

INSPECTION STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

- SYSTEMS AND COMPONENT PROBLEMS:
- ONE PRESSURIZER SAFETY VALVE REQUIRED REWORK AFTER THE RCS WAS PRESSURIZED TO 79 PSIG FOR LEAK TESTING.
- FACILITY ITEMS (PLANS AND PROCEDURES):
- REFUELING AND MODIFICATION OUTAGE COMMENCED 11/30/79; OUTAGE IS EXPECTED TO CONTINUE UNTIL AT LEAST MID-OCTOBER.

PLANT STATUS:

- IN COLD SHUTDOWN FOR MAJOR MODIFICATION ON THE FOLLOWING SAFETY-RELATED SYSTEMS. FINAL INSPECTION OF REINSTALLED BERGEN-PATTERSON LARGE BORE SNUBBERS; REFUELING WATER STORAGE TANK MODIFICATIONS; CHEMICAL ADDITION PUMPS FOR QUENCH SPRAY SYSTEM; INSPECTION/MODIFICATION OF PIPE HANGERS, SUPPORTS AND BASE PLATES; AND, MODIFICATION OF REACTOR COOLANT PUMP MOTOR THRUST RUNNERS.
- LAST IE SITE INSPECTION DATE: 9/13-30/80
- INSPECTION REPORT NO: 50-334/80-27

SUMMARY

INSPECTION SUMMARY

- 50-334/80-16 - JUN 1-30: ROUTINE INSPECTION BY TWO RESIDENT INSPECTORS (96 HRS) OF: LICENSEE ACTION ON PREVIOUS INSPECTION FINDINGS; PLANT OPERATIONS; IE BULLETIN FOLLOWUP; IN-OFFICE REVIEW OF LICENSEE EVENT REPORTS; LICENSEE EVENT FOLLOWUP; AND REPORTING REQUIREMENTS. THREE ITEMS OF NONCOMPLIANCE WERE IDENTIFIED: (1) FAILURE TO NOTIFY THE NRC REGARDING AN IMMEDIATE REPORTABLE EVENT; (2) FAILURE TO MAINTAIN PROCEDURES TO CONTROL OUT-OF-SERVICE SURVEY INSTRUMENTS; (3) FAILURE TO IMPLEMENT AND DOCUMENT CORRECTIVE ACTIONS IN RESPONSE TO NOTICES OF VIOLATION.
- 50-334/80-23 - AUG 4-7: ROUTINE, UNANNOUNCED INSPECTION BY THREE REGION-BASED INSPECTORS (80 HRS) OF THE OVERALL PREOPERATIONAL AND STARTUP TEST RESTART PROGRAM; OVERALL TRAINING PROGRAM INCLUDING GENERAL EMPLOYEE TRAINING, LICENSED OPERATOR REQUALIFICATION TRAINING, RADIATION WORKER TRAINING, CRAFTSMAN AND TECHNICIAN TRAINING; LICENSEE ACTION ON PREVIOUS INSPECTION FINDINGS; AND TOUR OF THE FACILITY. ONE ITEM OF NONCOMPLIANCE WAS IDENTIFIED - FAILURE TO DOCUMENT REVIEW OF ABNORMAL PROCEDURES BY ALL LICENSED OPERATORS.

REPORTS RECEIVED FROM LICENSEE

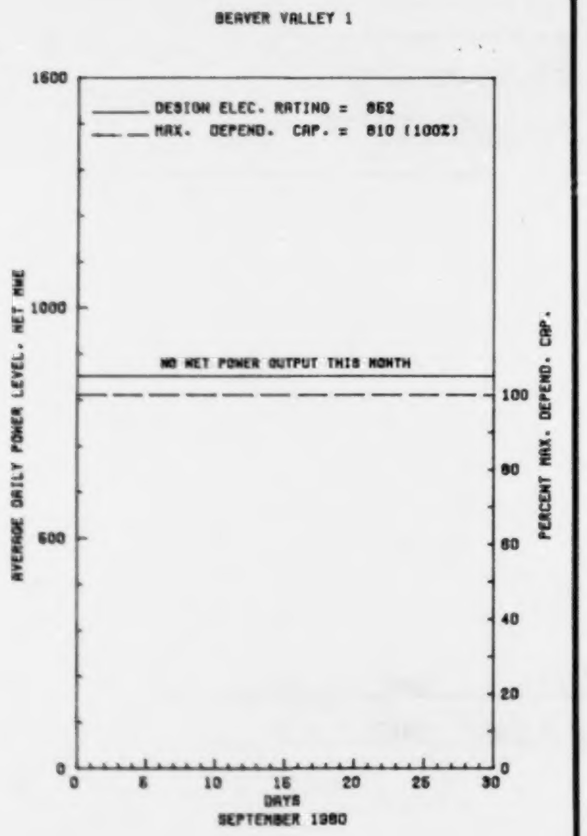
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-047/ 03L	08/05/80	09/04/80	EMERGENCY DIESEL GENERATOR NO. 1 OUTPUT CIRCUIT BREAKER FAILED TO CLOSE ONTO ENERGIZED EMERGENCY BUS DURING SURVEILLANCE TEST. RELAY REPLACED. WOULD NOT HAVE AFFECTED AUTO START.
80-059/ 01T	08/23/80	09/09/80	DIESEL DRIVEN FIRE PUMP STARTER MOTOR FAILED DUE TO SHORTED FIELD WINDING. STARTER MOTOR WAS REPLACED. TEMPORARY FIRE PUMP WAS AVAILABLE. MOTOR DRIVEN PUMP NOT AVAILABLE.
80-060/ 03L	08/27/80	08/28/80	POTENTIAL DAMAGE TO CHARGING PUMPS DUE TO LOW FLOW FOLLOWING STEAM LINE BREAK AND SAFETY INJECTION. VALVES TO BE CONTROLLED PROCEDURALLY AND MINIMUM FLOW ISOLATION SIGNAL REMOVED.
80-062/ 01T	09/08/80	09/09/80	OVERSTRESS CONDITION IDENTIFIED ON DISCHARGE LINE FROM THE BORON INJECTION TANK DURING SEISMIC REANALYSIS. PIPE SUPPORT TO BE MODIFIED PRIOR TO STARTUP.
80-063/ 01T	09/08/80	09/09/80	OVERSTRESS CONDITION IDENTIFIED DURING SEISMIC REANALYSIS ON DISCHARGE LINE FROM THE 1C CHARGING PUMP. PIPE SUPPORT TO BE MODIFIED PRIOR TO STARTUP.
80-064/ 03L	08/25/80	09/10/80	EXCEEDED 72 HOUR SURVEILLANCE REQUIREMENT ON REACTOR CHEMISTRY CHLORIDE AND FLUORIDE CONCENTRATIONS. SAMPLING PERFORMED. CONCENTRATIONS FOR PERIOD NEVER EXCEEDED SPECIFICATIONS.
80-065/ 01T	09/12/80	09/17/80	POTENTIAL SEISMIC OVERSTRESS ON BOTH REACTOR PLANT RIVER WATER SUPPLY LINES IDENTIFIED BY IEB 79-14 REVIEW. PIPE SUPPORT MODIFICATIONS COMPLETED PRIOR TO STARTUP.
80-066/ 01T	09/12/80	09/17/80	POTENTIAL SEISMIC OVERSTRESS ON "A" STEAM GENERATOR OUTLET LINE. IDENTIFIED BY IEB 79-14 REVIEW. PIPE SUPPORT MODIFICATIONS TO BE COMPLETED PRIOR TO STARTUP.
80-067/ 01T	09/12/80	09/17/80	POTENTIAL SEISMIC OVERSTRESS ON COMPONENT COOLING WATER HEADER FOR RESIDUAL HEAT REMOVAL SYSTEM IDENTIFIED BY IEB 79-14 RE-REVIEW. PIPE SUPPORT MODS COMPLETED BEFORE STARTUP.

REVIEWED BY OMPA LOEB: **DATE:** 11/4/80

REVIEWED BY NRR DL: W J Ross **DATE:** 11/6/80

BEAVER VALLEY 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-334
2. Utility Contact: DAVID R. TIMKO (412) 643-5308
3. Licensed Thermal Power (MWt): 2660
4. Nameplate Rating (Gross MWe): 1026 x 0.9 = 923
5. Design Electrical Rating (Net MWe): 852
6. Maximum Dependable Capacity (Gross MWe): 845
7. Maximum Dependable Capacity (Net MWe): 810
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	6,575.0	38,735.0
12. Hours Reactor Was Critical	.0	.0	13,774.7
13. Reactor Reserve Shutdown Hours	.0	.0	4,482.7
14. Hours Generator On-Line	.0	.0	13,105.0
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	0	26,974,253
17. Gross Electrical Energy (MWH)	0	0	8,277,940
18. Net Electrical Energy (MWH)	0	0	7,570,743
19. Unit Service Factor	.0	.0	34.0
20. Unit Availability Factor	.0	.0	34.0
21. Unit Capacity Factor (MDC Net)	.0	.0	27.2
22. Unit Capacity Factor (DER Net)	.0	.0	25.0
23. Unit Forced Outage Rate	.0	.0	46.4

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): MAJOR MODS OUTAGE (DECEMBER, 1979 THROUGH OCTOBER 18, 1980).
25. If Shut Down At End of Report Period, Estimated Date of Startup: 10/18/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	01/01/80	S	720.0	D	4		ZZ	ZZZZZZ	UNIT SHUTDOWN FOR MAJOR MODIFICATIONS AS REQUIRED BY THE NUCLEAR REGULATORY COMMISSION, INCLUDING NRC BULLETINS IEB 79-02 AND 79-14.

SUMMARY

BEAVER VALLEY 1 REMAINED SHUTDOWN FOR REQUIRED NRC SYSTEM MODIFICATIONS PER BULLETINS 79-02 AND 79-14.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

BIG ROCK POINT 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Michigan/Charlevoix 4 miles NE of Charlevoix
2.	Electric Reliability Council	East Central Area Reliability Coordination Agreement
3.	Type of Reactor	BWR
4.	Power Level (MWT)	240
5.	Design Electrical Rating (MWE Net)	72
6.	Date of Initial Criticality	September 27, 1962
7.	Date of Commercial Operation	March 29, 1963
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10.	Licensee	Consumers Power Company
11.	Corporate Address	212 West Michigan Avenue Jackson, Michigan 49201
12.	Corporate Contact	R. B. DeWitt
13.	Architect/Engineer	Bechtel Corporation
14.	Nuclear Steam Supply System	GE
15.	Constructor	Bechtel Corporation

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	D. C. BOYD
19.	IE Resident Inspector	G. Wright
20.	Licensing Project Manager	W. A. Paulson
21.	Docket Number	50-155
22.	License Number/Date of Issue	DPR-6 issued 8/30/62
23.	Public Document Room	Charlevoix Public Library 107 Clinton Street Charlevoix, Michigan

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

Operating routinely.

*LAST IE SITE INSPECTION DATE: August 4 through September 5, 1980

* INSPECTION REPORT NO: 80-10

SUMMARY

INSPECTION SUMMARY

* Inspection on August 4 through September 5, (80-10): Routine Resident inspector inspection of operational safety, bulletin review, circular review and LER 80-23 initial followup. The inspection involved 76 inspector hours onsite, including 8 hours off-shift inspection, by two NRC inspectors. No apparent items of noncompliance were identified in the areas inspected.

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-21/ 03L-0	07/29/80	08/25/80	Channel D RDS System out of service.
80-22/ 03L-0	08/07/80	09/05/80	Cells 38, 39 and 56 of Reactor Depress. battery had low readings.
80-23/ 01T-0	08/12/80	08/26/80	The design for steam line breaks in primary system indicates cont. atmosphere temp. can exceed temp. criteria.
80-24/ 03L-0	08/14/80	09/12/80	After auto. test circuit alarms, sensor Channel D of reactor depress. system was removed from service to vent the reactor water level transmitter.

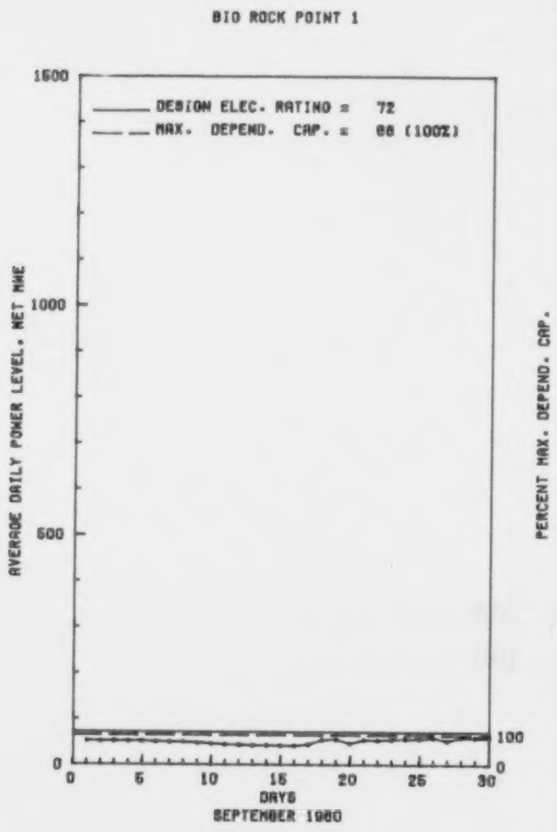
****

REVIEWED BY OMPA LOEB: John J. [illegible] **DATE:** 11-4-80

REVIEWED BY NRR DL: Walt A. [illegible] **DATE:** 11-5-80

BIG ROCK POINT 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-155
2. Utility Contact: G. Mimebauch (616) 547-6537 x180
3. Licensed Thermal Power (MWt): 240
4. Nameplate Rating (Gross MWe): 70.6 x 0.85 = 60
5. Design Electrical Rating (Net MWe): 72
6. Maximum Dependable Capacity (Gross MWe): 70
7. Maximum Dependable Capacity (Net MWe): 66
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

ITEM 21 CALCULATED WITH A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	6,575.0	153,498.0
12. Hours Reactor Was Critical	720.0	6,215.4	106,348.4
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	720.0	6,188.5	104,187.4
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	128,771	1,256,162	19,374,093
17. Gross Electrical Energy (MWH)	39,521	383,839	6,119,757
18. Net Electrical Energy (MWH)	37,350	362,656	5,790,997
19. Unit Service Factor	100.0	94.1	67.9
20. Unit Availability Factor	100.0	94.1	67.9
21. Unit Capacity Factor (MDC Net)	78.6	83.6	55.7
22. Unit Capacity Factor (DER Net)	72.0	76.6	52.4
23. Unit Forced Outage Rate	.0	.6	22.7

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUELING - 11/08/80
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
8	09/20/80	F	10.0	B	5		HA	PIPEXX	POWER REDUCTION - INTERMEDIATE PRESSURE TURBINE DRAIN LINE REPAIR.
9	09/27/80	F	3.1	A	5	80-33	CG	PUMPXX	POWER REDUCTION - TO TAKE CLEANUP PUMP OUT OF SERVICE FOR REPAIR.

SUMMARY

BIG ROCK POINT OPERATED NORMALLY FOR THE MONTH OF SEPTEMBER, WITH NO OUTAGES AND TWO REDUCTIONS FOR TURBINE DRAIN LINE AND CLEANUP PUMP REPAIR.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

BROWNS FERRY 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Alabama/Limestone 10 miles NW of Decatur
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	3293
5.	Design Electrical Rating (MWE Net)	1065
6.	Date of Initial Criticality	August 17, 1973
7.	Date of Commercial Operation	August 1, 1974
8.	Condenser Cooling Method	Once Through With Forced Draft Cooling Towers
9.	Condenser Cooling Water Source	Tennessee River

Utility & Contractor Information

10.	Licensee	Tennessee Valley Authority
11.	Corporate Address	500A Chestnut Street Tower II Chattanooga, Tennessee 37401
12.	Corporate Contact	H. G. Parris, Manager of Power.
13.	Architect/Engineer	TVA
14.	Nuclear Steam Supply System	GE
15.	Constructor	TVA

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. Lewis
18.	IE Section Chief	H. C. Dance
19.	IE Resident Inspector	R. Sullivan (205) 729-6196
20.	Licensing Project Manager	R. Clark
21.	Docket Number	50-259
22.	License Number/Date of Issue	DPR-33, 6/26/73
23.	Public Document Room	Athens Public Library South and Forrest Athens, Alabama 35611

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

100 PERCENT FULL POWER OPERATION.

+ LAST IE SITE INSPECTION DATE: AUGUST 11-13, 1980

+ INSPECTION REPORT NO: 50-259/80-33

SUMMARY

INSPECTION SUMMARY

+ INSPECTION JULY 1-31 (80-30): THIS ROUTINE INSPECTION INVOLVED 66 RESIDENT INSPECTOR-HOURS IN THE AREAS OF OPERATIONAL SAFETY, REPORTABLE OCCURRENCES, REACTOR TRIP FOLLOWUP, PLANT PHYSICAL PROTECTION, PREPARATIONS FOR REFUELING, PLANT CHEMISTRY AND TRAINING STAFF QUALIFICATIONS. IE BULLETIN FOLLOWUP. OF THE EIGHT AREAS INSPECTED NO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED.

INSPECTION AUGUST 11-13 (80-33): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 5 INSPECTOR-HOURS ON SITE IN THE AREAS OF FOLLOWUP ON ITEMS OF NONCOMPLIANCE RELATIVE TO 10 CFR PART 21 PROCEDURES AND PROCEDURE IMPLEMENTATION. THE PREVIOUSLY IDENTIFIED ITEMS OF NONCOMPLIANCE 50-259/79-25-01, 50-260/79-25-01, 50-296/79-25-01 AND INSPECTOR FOLLOWUP ITEM 50-259/79-25-02 REMAIN OPEN.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

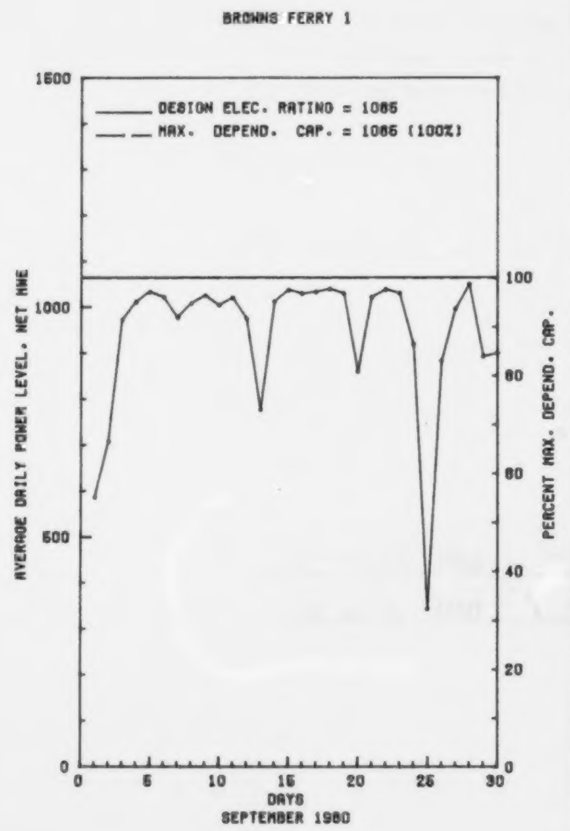
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-053/ 03L-0	07/04/80	08/01/80	EXCEEDED LHGR.
80-054/ 03L-0	07/06/80	07/18/80	RIVER WATER TEMPERATURE LIMITS REACHED.
80-054/ 01T	07/06/80	07/15/80	RIVER WATER TEMPERATURE HIGH.
80-057/ 03L-0	07/30/80	08/30/80	LIQUID PENETRANT TESTING MISSED LAST TWO REFUELING OUTAGES.
80-058/ 03L-0	07/28/80	08/28/80	TURBINE 1ST STAGE PRESSURE LIMIT SWITCH SETPOINT DRIFT.
80-059/ 03L-0	08/10/80	09/04/80	161KV LINE TRIPPED.

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 11-4-80

REVIEWED BY NRR DL: Dick Clark **DATE:** 11-06-80

BROWNS FERRY 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-259
2. Utility Contact: Don Green (205) 729-6846
3. Licensed Thermal Power (MWt): 3293
4. Nameplate Rating (Gross MWe): 1280 x 0.9 = 1152
5. Design Electrical Rating (Net MWe): 1065
6. Maximum Dependable Capacity (Gross MWe): 1098
7. Maximum Dependable Capacity (Net MWe): 1065
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	6,575.0	54,073.0
12. Hours Reactor Was Critical	703.3	4,356.3	32,633.6
13. Reactor Reserve Shutdown Hours	16.7	373.1	5,062.6
14. Hours Generator On-Line	697.2	4,221.5	31,834.4
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	2,175,862	12,430,818	86,567,311
17. Gross Electrical Energy (MWH)	699,320	3,997,100	28,515,560
18. Net Electrical Energy (MWH)	677,843	3,872,138	27,674,106
19. Unit Service Factor	96.8	64.2	58.9
20. Unit Availability Factor	96.8	64.2	58.9
21. Unit Capacity Factor (MDC Net)	88.4	55.3	48.1
22. Unit Capacity Factor (DER Net)	88.4	55.3	48.1
23. Unit Forced Outage Rate	3.2	9.9	30.6

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
143	09/01/80	F	10.0	A	3				TURBINE TRIP DUE TO STOP VALVE CLOSURE.
144	09/12/80	S	.0	H	5				DERATED FOR CONTROL ROD SEQUENCE EXCHANGE.
145	09/19/80	F	.0	A	5				DERATED FOR MAINTENANCE TO "B" REACTOR FEED PUMP.
146	09/20/80	F	.0	A	5				DERATED DUE TO LOSS OF UNIT PREFERRED POWER.
147	09/24/80	F	12.8	A	3				LOAD REJECT DUE TO NEGATIVE GROUND ON MAIN TRANSFORMER SUDDEN PRESSURE RELAY.

SUMMARY

BROWNS FERRY 1 OPERATED AT ABOUT 90% POWER, WITH THREE REDUCTIONS AND 2 OUTAGES LISTED IN DETAIL ABOVE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

BROWNS FERRY 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Alabama/Limestone 10 miles NW of Decatur
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	3293
5.	Design Electrical Rating (MWE Net)	1065
6.	Date of Initial Criticality	July 20, 1974
7.	Date of Commercial Operation	March 1, 1975
8.	Condenser Cooling Method	Once Through With Forced Draft Cooling Towers
9.	Condenser Cooling Water Source	Tennessee River

Utility & Contractor Information

10.	Licensee	Tennessee Valley Authority
11.	Corporate Address	500A Chestnut Street Tower II Chattanooga, Tennessee 37401
12.	Corporate Contact	H. G. Parris, Manager of Powe.
13.	Architect/Engineer	TVA
14.	Nuclear Steam Supply System	GE
15.	Constructor	TVA

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. Lewis
18.	IE Section Chief	H. C. Dance
19.	IE Resident Inspector	R. Sullivan (205) 729-6196
20.	Licensing Project Manager	R. Clark
21.	Docket Number	50-260
22.	License Number/Date of Issue	DPR-52, 6/28/74
23.	Public Document Room	Athens Public Library South and Forrest Athens, Alabama 35611

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

REACTOR IS IN COLD SHUTDOWN FOR REFUELING

+ LAST IE SITE INSPECTION DATE: AUGUST 11-13, 1980

+ INSPECTION REPORT NO: 50-260/80-26

SUMMARY

INSPECTION SUMMARY

+ INSPECTION JULY 1-31 (80-23): THIS ROUTINE INSPECTION INVOLVED 66 RESIDENT INSPECTOR-HOURS IN THE AREAS OF OPERATIONAL SAFETY, REPORTABLE OCCURRENCES, REACTOR TRIP FOLLOWUP, PLANT PHYSICAL PROTECTION, PREPARATIONS FOR REFUELING, PLANT CHEMISTRY AND TRAINING STAFF QUALIFICATIONS. IE BULLETIN FOLLOWUP. OF THE EIGHT AREAS INSPECTED NO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED.

INSPECTION AUGUST 11-13 (80-26): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 5 INSPECTOR-HOURS ON SITE IN THE AREAS OF FOLLOWUP ON ITEMS OF NONCOMPLIANCE RELATIVE TO 10 CFR PART 21 PROCEDURES AND PROCEDURE IMPLEMENTATION. THE PREVIOUSLY IDENTIFIED ITEMS OF NONCOMPLIANCE 50-259/79-25-01, 50-260/79-25-01, 50-296/79-25-01 AND INSPECTOR FOLLOWUP ITEM 50-259/79-25-02 REMAIN OPEN.

REPORTS RECEIVED FROM LICENSEE

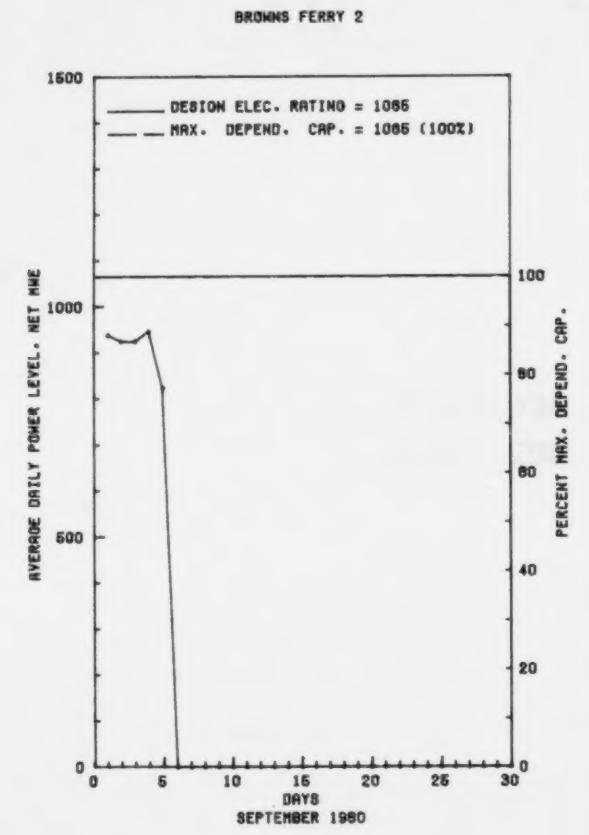
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-028/ 03L-0	07/15/80	08/13/80	CORE SPRAY DISCHARGE PRESSURE SWITCH SETPOINT DRIFT.
80-029/ 03L-0	07/22/80	08/20/80	REACTOR LOW WATER LEVEL SWITCH SETPOINT DRIFT.
80-030/ 03L-0	08/11/80	09/04/80	PRESSURE SWITCH REACTOR HIGH PRESSURE SETPOINT DRIFT.
80-031/ 01T	08/13/80	08/29/80	EXCEEDED MAXIMUM EXPOSURE TWO NODES FUEL BUNDLE 23-14.
80-032/ 03L-0	08/12/80	09/05/80	HIGH PRESSURE COOLANT INJECTION TRIP RUPTURE DISC FAILURE.

REVIEWED BY OMPA LOEB: John J. [illegible] **DATE:** 11-4-80

REVIEWED BY NRR DL: Dick Clark **DATE:** 11-06-80

BROWNS FERRY 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-260
2. Utility Contact: Don Green (205) 729-6846
3. Licensed Thermal Power (MWt): 3293
4. Nameplate Rating (Gross MWe): 1280 x 0.9 = 1152
5. Design Electrical Rating (Net MWe): 1065
6. Maximum Dependable Capacity (Gross MWe): 1098
7. Maximum Dependable Capacity (Net MWe): 1065
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
   NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
    NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	6,575.0	48,984.0
12. Hours Reactor Was Critical	118.8	5,304.1	29,788.5
13. Reactor Reserve Shutdown Hours	.0	649.0	12,423.9
14. Hours Generator On-Line	118.8	5,171.1	28,835.6
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	344,076	14,929,875	81,084,875
17. Gross Electrical Energy (MWH)	113,120	4,952,010	26,902,078
18. Net Electrical Energy (MWH)	107,818	4,807,248	26,138,717
19. Unit Service Factor	16.5	78.6	58.9
20. Unit Availability Factor	16.5	78.6	58.9
21. Unit Capacity Factor (MDC Net)	14.1	68.7	50.1
22. Unit Capacity Factor (DER Net)	14.1	68.7	50.1
23. Unit Forced Outage Rate	.0	12.2	33.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
    NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 11/02/80

II

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
155	09/05/80	S	601.2	C	2		RC	FUELXX	REFUELING OUTAGE COMMENCES.

SUMMARY

BROWNS FERRY 2 OPERATED UNTIL THE 5TH OF SEPTEMBER, WHEN REFUELING COMMENCED.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

BROWNS FERRY 3

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1. Location (state/county) (nearest population center): Alabama/Limestone, 10 miles NW of Decatur
2. Electric Reliability Council: Southeastern Electric Reliability Council
3. Type of Reactor: BWR
4. Power Level (MWT): 3293
5. Design Electrical Rating (MWE Net): 1065
6. Date of Initial Criticality: August 8, 1976
7. Date of Commercial Operation: March 1, 1977
8. Condenser Cooling Method: Once Through With Forced Draft Cooling Tower
9. Condenser Cooling Water Source: Tennessee River

Utility & Contractor Information

10. Licensee: Tennessee Valley Authority
11. Corporate Address: 500A Chestnut Street Tower II, Chattanooga, Tennessee 37401
12. Corporate Contact: H. G. Parris, Manager of Power
13. Architect/Engineer: TVA
14. Nuclear Steam Supply System: GE
15. Constructor: TVA

Regulatory Information

16. IE Region responsible: II
17. IE Branch Chief: R.C. Lewis
18. IE Section Chief: H. C. Dance
19. IE Resident Inspector: R. Sullivan (205) 729-6196
20. Licensing Project Manager: R. Clark
21. Docket Number: 50-296
22. License Number/Date of Issue: DPR-68, 7/2/76
23. Public Document Room: Athens Public Library, South and Forrest, Athens, Alabama 35611

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

FULL POWER OPERATIONS.

+ LAST IE SITE INSPECTION DATE: AUGUST 11-13, 1980

+ INSPECTION REPORT NO: 50-296/80-27

SUMMARY

INSPECTION SUMMARY

+ INSPECTION JULY 1-31 (80-24): THIS ROUTINE INSPECTION INVOLVED 66 RESIDENT INSPECTOR-HOURS IN THE AREAS OF OPERATIONAL SAFETY, REPORTABLE OCCURRENCES, REACTOR TRIP FOLLOWUP, PLANT PHYSICAL PROTECTION, PREPARATIONS FOR REFUELING, PLANT CHEMISTRY AND TRAINING STAFF QUALIFICATIONS. IE BULLETIN FOLLOWUP. OF THE EIGHT AREAS INSPECTED NO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED.

INSPECTION AUGUST 11-13 (80-27): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 4 INSPECTOR-HOURS ON SITE IN THE AREAS OF FOLLOWUP ON ITEMS OF NONCOMPLIANCE RELATIVE TO 10 CFR PART 21 PROCEDURES AND PROCEDURE IMPLEMENTATION. THE PREVIOUSLY IDENTIFIED ITEMS OF NONCOMPLIANCE 50-259/79-25-01, 50-260/79-25-01, 50-296/79-25-01 AND INSPECTOR FOLLOWUP ITEM 50-259/79-25-02 REMAIN OPEN.

REPORTS RECEIVED FROM LICENSEE

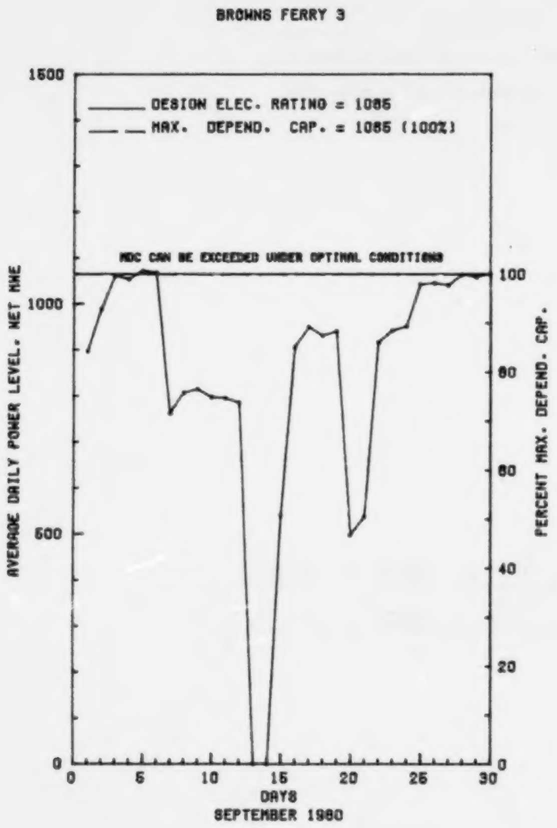
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-025/03L-0	06/25/80	07/24/80	STRIP HEATER DAMAGED CAUSE FALSE ALARMS.
80-028/03L-0	08/06/80	08/28/80	REACTOR VESSEL WATER LEVEL SWITCH ABOVE SETPOINT.
80-029/03L-0	08/02/80	09/04/80	COMPUTER IMPROPERLY SET GIVING WRONG VALUES.

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 11-4-80

REVIEWED BY NRR DL: Dick Clark **DATE:** 11-06-80

BROWNS FERRY 3

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-296
2. Utility Contact: Don Green (205) 729-6846
3. Licensed Thermal Power (MWt): 3293
4. Nameplate Rating (Gross MWe): 1280 x 0.9 = 1152
5. Design Electrical Rating (Net MWe): 1065
6. Maximum Dependable Capacity (Gross MWe): 1098
7. Maximum Dependable Capacity (Net MWe): 1065
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	6,575.0	31,439.9
12. Hours Reactor Was Critical	669.0	5,769.2	24,676.9
13. Reactor Reserve Shutdown Hours	50.9	152.6	1,811.0
14. Hours Generator On-Line	654.3	5,657.4	24,095.2
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,922,993	17,411,179	70,519,045
17. Gross Electrical Energy (MWH)	625,700	5,843,150	23,245,730
18. Net Electrical Energy (MWH)	607,023	5,680,579	22,568,352
19. Unit Service Factor	90.9	86.0	76.6
20. Unit Availability Factor	90.9	86.0	76.6
21. Unit Capacity Factor (MDC Net)	79.2	81.1	67.4
22. Unit Capacity Factor (DER Net)	79.2	81.1	67.4
23. Unit Forced Outage Rate	9.1	14.0	10.2

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUEL OUTAGE - OCTOBER 31, 1980.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
143	09/01/80	F	.0	A	5				DERATED WHEN "A" RECIRCULATION PUMP TRIPPED.
144	09/07/80	S	.0	B	5				DERATED FOR TURBINE CONTROL VALVE TESTS AND SI'S.
145	09/07/80	F	.0	A	5				DERATED WHEN MSIV 1-26 CLOSED TOO FAST.
146	09/12/80	F	41.2	A	2				MAINTENANCE ON MSIV 1-26.
147	09/14/80	F	9.5	A	9				MAINTENANCE ON FEEDWATER VALVE (3-219A) LEAK, REACTOR WAS ALREADY DOWN.
148	09/20/80	F	15.0	A	3				LOSS OF PREFERRED POWER WHEN "A" 480V SHUTDOWN BOARD FAILED TO TRANSFER BACK TO NORMAL CAUSING LOSS OF "A" RPS MG SET.

SUMMARY

BROWNS FERRY 3 OPERATED AT ABOUT 90% POWER FOR THE MONTH, WITH THREE REDUCTIONS AND THREE OUTAGES LISTED IN DETAIL ABOVE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

NRC POWER RESTRICTIONS: NONE

BRUNSWICK 1

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	North Carolina/Brunswick 3 miles N of Southport
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2436
5.	Design Electrical Rating (MWE Net)	821
6.	Date of Initial Criticality	October 8, 1976
7.	Date of Commercial Operation	March 18, 1977
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Cape Fear River

Utility & Contractor Information

10.	Licensee	Carolina Power and Light Company
11.	Corporate Address	411 Fayetteville Street Raleigh, North Carolina 27602
12.	Corporate Contact	J. A. Jones Executive Vice President
13.	Architect/Engineer	United Engineers & Constructors, Inc.
14.	Nuclear Steam Supply System	GE
15.	Constructor	Brown and Root, Inc.

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R.C. Lewis
18.	IE Section Chief	P.J. Kellogg
19.	IE Resident Inspector	J. E. Outzs (919) 457-9531
20.	Licensing Project Manager	J. Hannon
21.	Docket Number	50-325
22.	License Number/Date of Issue	DPR-71, 9/8/76
23.	Public Document Room	Southport-Brunswick County Library 109 W. Moore Street Southport, North Carolina 28461

INSPECTION STATUS

ENFORCEMENT STATUS

AS REQUIRED BY 10 CFR 50, APPENDIX B, CRITERION V, AND IMPLEMENTED BY SECTION 6.8 OF THE TECHNICAL SPECIFICATION "WRITTEN PROCEDURES SHALL BE ESTABLISHED, IMPLEMENTED, AND MAINTAINED...AS RECOMMENDED IN APPENDIX A OF REGULATORY GUIDE 1.33, NOV. 1972." CONTRARY TO THE ABOVE, CP&L'S SPECIAL TEST PROCEDURE SP-79-22 "INSPECTION AND TESTING PROCEDURE FOR CONCRETE EXPANSION ANCHORS" AND RESPONSE TO IE BULLETIN 79-14, SERIAL: GD-79-1844 DATED JUNE 23, 1979 FAILED TO LIST THE CONTROL ROD DRIVE (CRD) AND RECIRCULATION LINES AS SAFETY RELATED AND DID NOT PERFORM ANY INSPECTIONS AS REQUIRED BY BULLETINS 79-02 AND 79-14. (8032 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

ROUTINE POWER OPERATION

LAST IE SITE INSPECTION DATE: AUGUST 1-31, 1980

INSPECTION REPORT NO: 50-325/80-35

SUMMARY

INSPECTION SUMMARY

INSPECTION JULY 1-31 (80-33): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 65 INSPECTOR-HOURS ON SITE IN THE AREAS OF CONTROL ROOM OBSERVATIONS, REVIEW OF PLANT LOGS, TOUR OF VITAL AREAS, PHYSICAL SECURITY BARRIERS, RADIATION PROTECTION, HOUSEKEEPING AND CLEANLINESS, FIRE PROTECTION, INSPECTION OF RADWASTE SHIPMENT, REVIEW OF ORGANIZATIONAL CHANGES, FOLLOWUP ON IE BULLETINS 80-17 AND 80-18, REVIEW OF LICENSEE EVENT REPORTS (LER'S) AND LICENSEE'S RESPONSE TO IE CIRCULARS. THIS ROUTINE INSPECTION BY THE RESIDENT INSPECTORS INVOLVED 65 MANHOURS OF INSPECTION TIME ON SITE IN THE TWELVE AREAS IDENTIFIED ABOVE. OF THE TWELVE AREAS INSPECTED NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION JULY 27 - AUGUST 1 (80-34): INSPECTED CHEM-NUCLEAR WASTE DISPOSAL SITE, BARNWELL, SOUTH CAROLINA FOR UNANNOUNCED INSPECTIONS OF LICENSEE-SHIPPERS. EACH LICENSEE-SHIPPER VEHICLE WAS INSPECTED FOR COMPLIANCE WITH DEPARTMENT OF TRANSPORTATION (DOT) AND NUCLEAR REGULATORY COMMISSION (NRC) REGULATIONS AS FOLLOWS: (1) SHIPPING PAPER REQUIREMENTS; (2) DOT PLACARDING REQUIREMENTS; (3) RADIATION LEVELS; (4) REMOVABLE CONTAMINATION; (5) DOT MARKING AND LABELING REQUIREMENTS FOR PACKAGES; (6) DOT AND NRC REQUIREMENTS FOR EXTERNAL PACKAGE FEATURES, AND (7) PROHIBITED ARTICLES OR CONTENTS. OF THE SEVEN AREAS INSPECTED INVOLVING 98 SHIPMENTS, NO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED.

INSPECTION AUGUST 1-31 (80-35): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 55 INSPECTOR-HOURS ON SITE IN THE AREAS OF CONTROL ROOM OBSERVATIONS, REVIEW OF PLANT LOGS, TOURS OF VITAL AREAS, TOURS OF PHYSICAL SECURITY BARRIERS, RADIATION PROTECTION, FOLLOWUP ON IE BULLETIN 80-17, REVIEW OF LICENSEE EVENT REPORTS (LERS) AND INDEPENDENT INSPECTION EFFORT. THIS ROUTINE INSPECTION BY THE RESIDENT INSPECTOR INVOLVED 30 MANHOURS OF INSPECTION TIME ON SITE IN THE TEN AREAS IDENTIFIED ABOVE. OF THE TEN AREAS INSPECTED NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

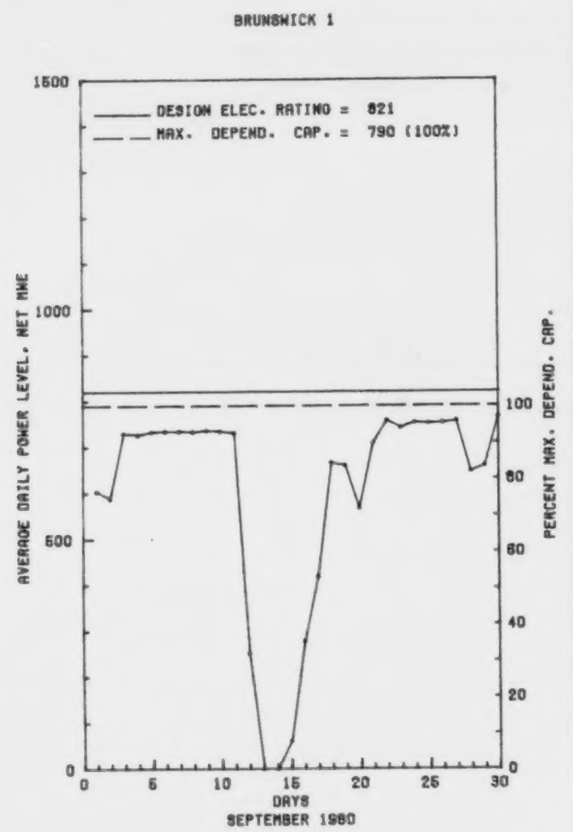
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-017/01T	09/06/80	09/10/80	NO. 1 HPCI STEAM SUPPLY LEAK REQUIRED REMOVAL FROM SERVICE.
80-046/01T	07/11/80	07/14/80	SCRAM DISCHARGE PIPING SUPPORT MODIFICATIONS REQUIRED.
80-055/03L-0	08/12/80	09/05/80	VESSEL WATER CONDUCTIVITY LIMITS EXCEEDED.
80-059/01T	07/16/80	07/17/80	NONCONSERVATIVE MAXIMUM TOTAL PEAKING FACTOR DUE TO ERROR IN FUEL LENGTH.
80-060/03L-0	07/15/80	08/12/80	CORE SPRAY DISCHARGE PRESSURE TRANSMITTER OUT OF CALIBRATION.
80-066/03L-0	08/24/80	09/12/80	FAILED CONTAINMENT OXYGEN MONITOR.

REVIEWED BY OMPA LOEB: John J. Ruzzo **DATE:** 11-4-80

REVIEWED BY NRR DL: John N Hannon **DATE:** 11-5-80

BRUNSWICK 1

AVERAGE POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-325
2. Utility Contact: E. Willis (919) 457-9521
3. Licensed Thermal Power (MWt): 2436
4. Nameplate Rating (Gross MWe): 963 x 0.9 = 867
5. Design Electrical Rating (Net MWe): 821
6. Maximum Dependable Capacity (Gross MWe): 815
7. Maximum Dependable Capacity (Net MWe): 790
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	6,575.0	31,032.0
12. Hours Reactor Was Critical	720.0	4,316.9	21,706.5
13. Reactor Reserve Shutdown Hours	.0	.0	1,647.1
14. Hours Generator On-Line	649.1	3,964.3	20,297.1
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,347,267	7,775,423	41,684,919
17. Gross Electrical Energy (MWH)	446,687	2,598,537	13,837,356
18 Net Electrical Energy (MWH)	430,450	2,496,885	13,304,695
19. Unit Service Factor	90.2	60.3	65.4
20. Unit Availability Factor	90.2	60.3	65.4
21. Unit Capacity Factor (MDC Net)	75.7	48.1	54.3
22. Unit Capacity Factor (DER Net)	72.8	46.3	52.2
23. Unit Forced Outage Rate	9.8	10.2	19.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
021	09/12/80	S	70.9	B	1		HA	GENERA	SEPARATED FROM GRID TO INVESTIGATE AND/OR REPAIR A BEARING ON THE GENERATOR. THE BEARING WAS REPLACED WITH A NEW BEARING. THE EXCITER COUPLING WAS REALIGNED.

SUMMARY

BRUNSWICK 1 OPERATED AT APPROXIMATELY 90% POWER FOR THE MONTH, WITH ONE OUTAGE TO REPLACE A GENERATOR BEARING.

LEGEND

[1] Type:
F-Forced
S-Scheduled

[2] Reason:
A-Equipment Failure
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error
H-Other

[3] Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Continuations
5-Load Reduction
9-Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

BRUNSWICK 2

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	North Carolina/Brunswick 3 miles N of Southport
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2436
5.	Design Electrical Rating (MWE Net)	821
6.	Date of Initial Criticality	March 20, 1975
7.	Date of Commercial Operation	November 3, 1975
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Cape Fear River

Utility & Contractor Information

10.	Licensee	Carolina Power and Light Company
11.	Corporate Address	411 Fayetteville Street Raleigh, North Carolina 27602
12.	Corporate Contact	J. A. Jones Executive Vice President
13.	Architect/Engineer	United Engineers & Constructors, Inc.
14.	Nuclear Steam Supply System	GE
15.	Constructor	Brown and Root, Inc.

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R.C. Lewis
18.	IE Section Chief	P.J. Kellogg
19.	IE Resident Inspector	J. E. Outzs (919) 457-9631
20.	Licensing Project Manager	J. Hannon
21.	Docket Number	50-324
22.	License Number/Date of Issue	DPR-62, 12/27/74
23.	Public Document Room	Southport-Brunswick County Library 109 W. Moore Street Southport, North Carolina 28461

INSPECTION STATUS

ENFORCEMENT STATUS

AS REQUIRED BY 10 CFR 50, APPENDIX B, CRITERION V, AND IMPLEMENTED BY SECTION 6.8 OF THE TECHNICAL SPECIFICATION "WRITTEN PROCEDURES SHALL BE ESTABLISHED, IMPLEMENTED, AND MAINTAINED...AS RECOMMENDED IN APPENDIX A OF REGULATORY GUIDE 1.33, NOV. 1972." CONTRARY TO THE ABOVE, CP&L'S SPECIAL TEST PROCEDURE SP-79-22 "INSPECTION AND TESTING PROCEDURE FOR CONCRETE EXPANSION ANCHORS" AND RESPONSE TO IE BULLETIN 79-14, SERIAL: GD-79-1844 DATED JUNE 23, 1979 FAILED TO LIST THE CONTROL ROD DRIVE (CRD) AND RECIRCULATION LINES AS SAFETY RELATED AND DID NOT PERFORM ANY INSPECTIONS AS REQUIRED BY BULLETINS 79-02 AND 79-14.
(8029 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

+ UNIT IN NORMAL OPERATION.

+ LAST IE SITE INSPECTION DATE: AUGUST 1-31, 1980

+ INSPECTION REPORT NO: 50-324/80-32

SUMMARY

INSPECTION SUMMARY

+ INSPECTION JULY 1-31 (80-30): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 129 INSPECTOR-HOURS ON SITE IN THE AREAS OF CONTROL ROOM OBSERVATIONS, REVIEW OF PLANT LOGS, TOUR OF VITAL AREAS, PHYSICAL SECURITY BARRIERS, RADIATION PROTECTION, HOUSEKEEPING AND CLEANLINESS, FIRE PROTECTION, INSPECTION OF RADWASTE SHIPMENT, REVIEW OF ORGANIZATIONAL CHANGES, FOLLOWUP ON IE BULLETINS 80-17 AND 80-18, REVIEW OF LICENSEE EVENT REPORTS (LER'S) AND LICENSEE'S RESPONSE TO IE CIRCULARS. THIS ROUTINE INSPECTION BY THE RESIDENT INSPECTOR INVOLVED 64 MANHOURS OF INSPECTION TIME ON SITE IN THE TWELVE AREAS IDENTIFIED ABOVE. OF THE TWELVE AREAS INSPECTED NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION JULY 27 - AUGUST 1 (80-31): INSPECTED CHEM-NUCLEAR WASTE DISPOSAL SITE, BARNWELL, SOUTH CAROLINA FOR UNANNOUNCED INSPECTIONS OF LICENSEE-SHIPPERS. EACH LICENSEE-SHIPPER VEHICLE WAS INSPECTED FOR COMPLIANCE WITH DEPARTMENT OF TRANSPORTATION (DOT) AND NUCLEAR REGULATORY COMMISSION (NRC) REGULATIONS AS FOLLOWS: (1) SHIPPING PAPER REQUIREMENTS; (2) DOT PLACARDING REQUIREMENTS; (3) RADIATION LEVELS; (4) REMOVABLE CONTAMINATION; (5) DOT MARKING AND LABELING REQUIREMENTS FOR PACKAGES; (6) DOT AND NRC REQUIREMENTS FOR EXTERNAL PACKAGE FEATURES, AND (7) PROHIBITED ARTICLES OR CONTENTS. OF THE SEVEN AREAS INSPECTED INVOLVING 98 SHIPMENTS, NO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED.

INSPECTION AUGUST 1-31 (80-32): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 55 INSPECTOR-HOURS ON SITE IN THE AREAS OF CONTROL ROOM OBSERVATIONS, REVIEW OF PLANT LOGS, TOURS OF VITAL AREAS, TOURS OF PHYSICAL SECURITY BARRIERS, RADIATION PROTECTION, FOLLOWUP ON IE BULLETIN 80-17, REVIEW OF LICENSEE EVENT REPORTS (LERS) AND INDEPENDENT INSPECTION EFFORT. THIS ROUTINE INSPECTION BY THE RESIDENT INSPECTOR INVOLVED 25 MANHOURS OF INSPECTION TIME ON SITE IN THE TEN AREAS IDENTIFIED ABOVE. OF THE TEN AREAS INSPECTED NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

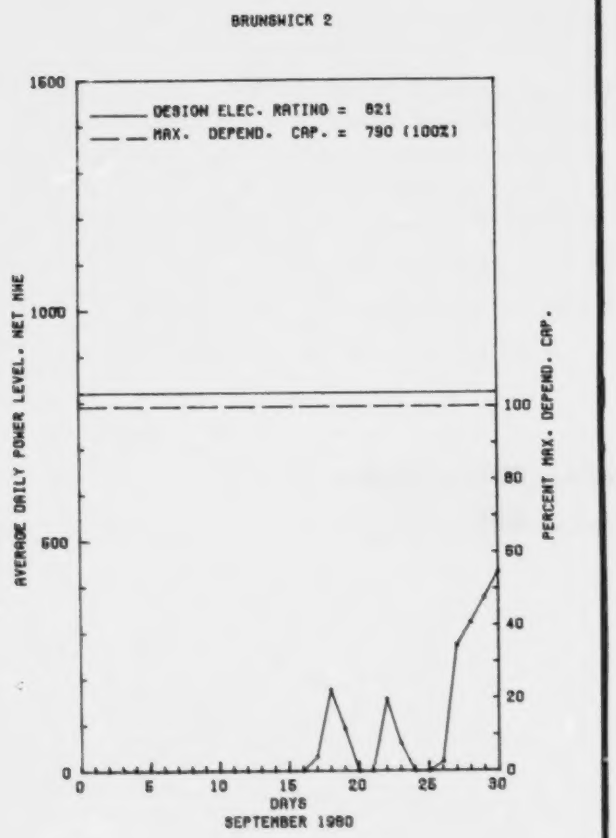
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-046/01T-0	07/11/80	07/14/80	SCRAM DISCHARGE PIPING SUPPORT MODIFICATIONS REQUIRED.
80-050/03L-0	08/21/80	09/10/80	REACTOR LOW PRESSURE SWITCH OUT OF CALIBRATION.
80-057/03L-0	07/28/80	08/26/80	REACTOR LOW WATER LOW LEVEL SWITCH 2-B21-LIS-NO-31C-3 OUT OF CALIBRATION.
80-059/01T-0	07/16/80	07/17/80	NONCONSERVATIVE MAXIMUM TOTAL PEAKING FACTOR DUE TO ERROR IN FUEL LENGTH.
80-062/01T-0	09/07/80	09/08/80	BOTH RCIC AND HPCI TURBINE EXHAUST WERE FOUND CLOSED.

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 11-4-80

REVIEWED BY NRR DL: John F. Hannon **DATE:** 11-5-80

BRUNSWICK 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-324
2. Utility Contact: E. Willis (919) 457-9521
3. Licensed Thermal Power (MWt): 2436
4. Nameplate Rating (Gross MWe): 963 x 0.9 = 867
5. Design Electrical Rating (Net MWe): 821
6. Maximum Dependable Capacity (Gross MWe): 815
7. Maximum Dependable Capacity (Net MWe): 790
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	6,575.0	43,056.0
12. Hours Reactor Was Critical	485.8	1,851.5	27,175.4
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	178.1	1,406.2	25,280.2
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	150,571	2,755,531	46,489,993
17. Gross Electrical Energy (MWH)	50,685	923,298	15,538,372
18. Net Electrical Energy (MWH)	39,848	881,974	14,922,446
19. Unit Service Factor	24.7	21.4	58.7
20. Unit Availability Factor	24.7	21.4	58.7
21. Unit Capacity Factor (MDC Net)	7.0	17.0	43.9
22. Unit Capacity Factor (DER Net)	6.7	16.3	42.2
23. Unit Forced Outage Rate	45.0	15.3	13.1

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
06	03/01/80	S	396.2	B	2		RC	FUELXX	THE UNIT WAS SEPARATED FROM THE GRID FOR A PLANNED REFUELING /MAINTENANCE OUTAGE, WHICH WAS EXTENDED BEYOND ITS SCHEDULED DATE BECAUSE OF UNANTICIPATED MAINTENANCE AND REGULATORY PROBLEMS.
07	09/17/80	S	.2	B	1		ED	TURBIN	REMOVED THE TURBINE FROM GRID FOR A TURBINE OVERSPEED TRIP TEST, REQUIRED AFTER EXTENSIVE MAINTENANCE ON THE TURBINE. NO CORRECTIVE ACTION REQUIRED.
08	09/19/80	F	68.6	A	3		IA	ZZZZZZ	THE REACTOR SCRAMMED DUE TO CAUSES UNKNOWN. NO TEST MAINTENANCE, OR OTHER WORK WAS IN PROGRESS WHICH COULD EXPLAIN THE SCRAM. ALL REACTOR PARAMETERS WERE NORMAL UP UNTIL THE TIME OF THE SCRAM. NO REASON FOR THE SCRAM COULD BE DETERMINED. THE OUTAGE WAS EXTENDED DUE TO THE REPLACEMENT OF THE PILOT ASSEMBLIES ON SAFETY RELIEF VALVES F013B AND F013K. BOTH HAD BEEN SHOWING INDICATION OF PILOT LEAKAGE.
009	09/23/80	F	76.9	A	1		CE	VALVEX	THE UNIT WAS SEPARATED FROM THE GRID DUE TO HIGH DRYWELL FLOOR DRAIN LEAKAGE. THE LEAKAGE WAS CAUSED BY A PACKING LEAK ON VALVE E51-F007. THE VALVE WAS REPACKED. WHEN STARTUP WAS ATTEMPTED A LEAK WAS DISCOVERED ON FW-FV46. A DRAIN LINE FROM THE VALVE BODY HAD A CRACKED WELD. THE CRACKED WELD WAS REPAIRED.

SUMMARY

BRUNSWICK 2 RETURNED ON-LINE FROM AN EXTENSIVE REFUELING /MAINTENANCE OUTAGE ON THE 17TH, AND INCURRED THREE ADDITIONAL OUTAGES FOR REPAIRS AND TESTING.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

CALVERT CLIFFS 1

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Maryland/Calvert 40 miles S of Annapolis
2.	Electric Reliability Council	Mid-Atlantic Area Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2700
5.	Design Electrical Rating (MWE Net)	845
6.	Date of Initial Criticality	October 7, 1974
7.	Date of Commercial Operation	May 8, 1975
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Chesapeake Bay

Utility & Contractor Information

10.	Licensee	Baltimore Gas and Electric Company
11.	Corporate Address	P. O. Box 1475 Baltimore, Maryland 21203
12.	Corporate Contact	A. E. Lundvall, Jr. Vice President-Supply
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Combustion Engineering
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Section Chief	E. C. McCabe, Jr.
19.	IE Resident Inspector	R. Architzel (301) 586-2627
20.	Licensing Project Manager	E. L. Conner, Jr.
21.	Docket Number	50-317
22.	License Number/Date of Issue	DPR-53, 7/31/74
23.	Public Document Room	Calvert County Library Prince Frederick, Maryland 20678

INSPECTION STATUS

ENFORCEMENT STATUS

FAILURE TO MAINTAIN POSITIVE ACCESS CONTROL TO PROTECTED AREA.
(8012 INFRACTION)

FAILURE TO CONTROL ACCESS TO VITAL AREA.
(8012 INFRACTION)

FAILURE TO MAINTAIN CONTINUOUS COMMUNICATIONS.
(8012 INFRACTION)

FAILURE TO ISSUE WRITTEN PROCEDURE.
(8012 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS:

+ END OF CYCLE COASTDOWN BEGAN 9/25. REFUELING BEGINS 10/17.

+ LAST IE SITE INSPECTION DATE: 9/15-19/80

+ INSPECTION REPORT NO: 50-317/80-17

SUMMARY

INSPECTION SUMMARY

+ NO REPORTS ISSUED.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

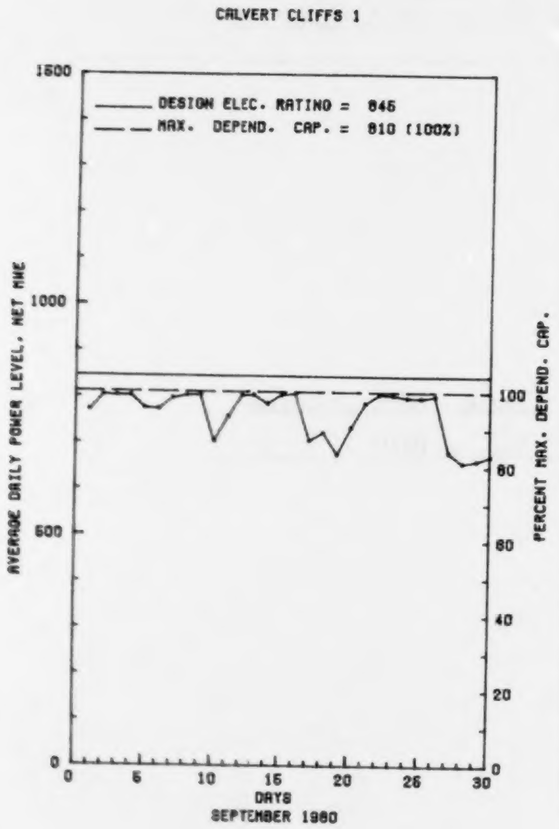
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-038/ 03L	08/01/80	08/29/80	CONTROL ROOM AIR CONDITIONING UNIT OUT OF SERVICE DUE TO HIGH LOAD. UNLOADER RESET TO RETURN UNIT TO SERVICE.
80-039/ 03L	08/15/80	09/15/80	NO. 12 CONTROL ROOM AIR CONDITIONER TRIPPED DUE TO HIGH DISCHARGE PRESSURE. SYSTEM DESIGN BEING MODIFIED FOR PERMANENT CORRECTIVE ACTION.
80-040/ 03L	08/06/80	09/06/80	EMERGENCY SALTWATER DISCHARGE VALVE OPENED INADVERTENTLY, REDUCING HEADER PRESSURE.
80-042/ 03L	08/06/80	09/06/80	ESFAS ACTUATION CHANNEL ZE INOPERABLE FOR CORRECTIVE MAINTENANCE TO REPLACE DEFECTIVE CHANNEL B CONTAINMENT ISOL SYSTEM ISOLATION MODULE.
80-043/ 03L	08/19/80	09/08/80	500 KV BLACK BUS OUT OF SERVICE TO REPAIR DISCONNECT DEVICE.
80-044/ 03L	08/20/80	09/08/80	CONTAINMENT SUMP DRAIN VALVES INADVERTENTLY LEFT OPEN.
80-045/ 03L	08/20/80	09/20/80	PLANT COMPUTER OOS (AGAIN ON 8/21) DUE TO LOOSE CONNECTOR. REACTOR POWER DECREASED PER TS.
80-046/ 03L	08/16/80	09/16/80	CONTAINMENT RADIATION MONITORING SYSTEM OOS FOR 15 MINUTES.
80-048/ 01T	09/03/80	09/17/80	BOTH CONTROL ROOM AIR CONDITIONERS OOS WHEN NO. 12 CRAC WAS STOPPED TO TEST NO. 11 UNIT AND NO. 11 DID NOT START.

REVIEWED BY OMPA LOEB: Beebe **DATE:** 11/04/80

REVIEWED BY NRR DL: E L Conner **DATE:** 11/5/80

CALVERT CLIFFS 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-317
2. Utility Contact: S. D. Merson (301) 234-5240
3. Licensed Thermal Power (MWt): 2700
4. Nameplate Rating (Gross MWe): 1020 x 0.9 = 918
5. Design Electrical Rating (Net MWe): 845
6. Maximum Dependable Capacity (Gross MWe): 845
7. Maximum Dependable Capacity (Net MWe): 810
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	6,575.0	47,340.0
12. Hours Reactor Was Critical	720.0	6,011.0	38,450.6
13. Reactor Reserve Shutdown Hours	.0	38.3	1,098.8
14. Hours Generator On-Line	720.0	5,961.6	37,643.6
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,858,080	14,306,193	89,643,181
17. Gross Electrical Energy (MWH)	576,267	4,493,216	29,305,746
18. Net Electrical Energy (MWH)	549,582	4,271,270	27,920,476
19. Unit Service Factor	100.0	90.4	79.5
20. Unit Availability Factor	100.0	90.4	79.5
21. Unit Capacity Factor (MDC Net)	94.2	80.2	72.8
22. Unit Capacity Factor (DER Net)	90.3	76.9	69.8
23. Unit Forced Outage Rate	.0	3.1	8.1

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUELING, MAINT., INSPECT. & TMI MODS. - 10/17-- 7-8 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
NONE									

SUMMARY

CALVERT CLIFFS 1 OPERATED NORMALLY IN SEPTEMBER WHILE IN THE PROCESS OF COASTING DOWN FOR A PLANNED OCTOBER REFUELING.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

CALVERT CLIFFS 2

NRC OWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Maryland/Calvert 40 miles S of Annapolis
2.	Electric Reliability Council	Mid-Atlantic Area Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2700
5.	Design Electrical Rating (MWE Net)	845
6.	Date of Initial Criticality	November 30, 1976
7.	Date of Commercial Operation	April 1, 1977
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Chesapeake Bay

Utility & Contractor Information

10.	Licensee	Baltimore Gas and Electric Company
11.	Corporate Address	P. O. Box 1475 Baltimore, Maryland 21203
12.	Corporate Contact	A. E. Lundvall, Jr. Vice President-Supply
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Combustion Engineering
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	I E Section Chief	E. C. MCCABE, JR.
19.	IE Resident Inspector	R. Architzel (301) 586-2627
20.	Licensing Project Manager	E. L. Conner, Jr.
21.	Docket Number	50-318
22.	License Number/Date of Issue	DPR-69, 8/13/76
23.	Public Document Room	Calvert County Library Prince Frederick, Maryland 20678

INSPECTION STATUS

ENFORCEMENT STATUS

FAILURE TO MAINTAIN POSITIVE ACCESS CONTROL TO PROTECTED AREA.
(8011 INFRACTION)

FAILURE TO CONTROL ACCESS TO VA.
(8011 INFRACTION)

FAILURE TO MAINTAIN CONTINUOUS COMMUNICATIONS.
(8011 INFRACTION)

FAILURE TO ISSUE WRITTEN PROCEDURES.
(8011 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS:

No significant new information.

+ LAST IE SITE INSPECTION DATE: 9/15-19/80

+ INSPECTION REPORT NO: 50-318/80-16

SUMMARY

INSPECTION SUMMARY

+ NO REPORTS ISSUED.

REPORTS RECEIVED FROM LICENSEE

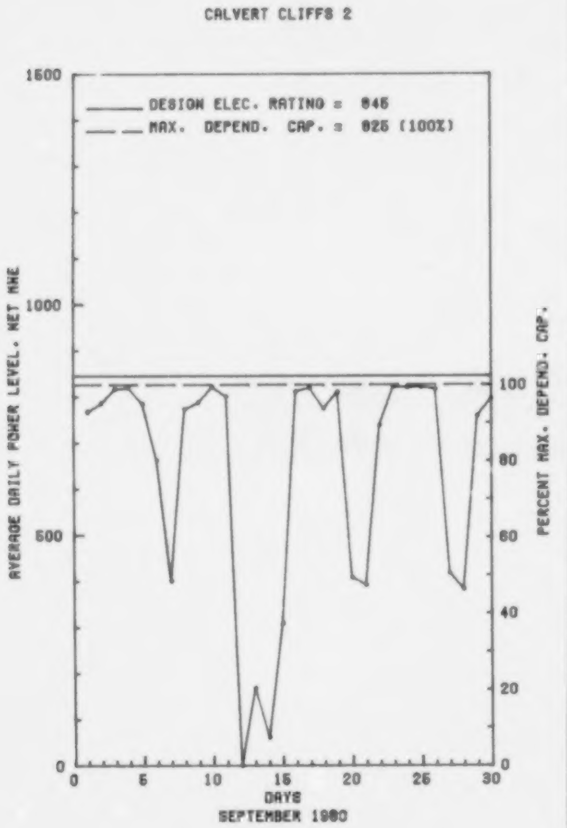
```
=====================================================================================================
 NUMBER    DATE OF    DATE OF    SUBJECT
            EVENT      REPORT
=====================================================================================================
 80-036/   07/29/80   08/29/80   ESF CONTAINMENT ISOLATION SYSTEM BISTABLES FOUND OSCILLATING.
  03L                              COMPONENTS REPLACED AND SENT TO MANUFACTURER FOR INVEST.
 80-037/   08/18/80   09/18/80   STEAM GENERATOR BLOWDOWN PH BELOW SPECIFICATION FOR 2 HOURS. CAUSED
  03L                              BY TECHNICIAN FAILURE TO ASSURE PROPER CHEMICAL ADDITION LINEUP.

 ==========================================================================================================
```

REVIEWED BY OMPA LOEB: M R Beebe DATE: 11/04/80

REVIEWED BY NRR DL: E L Conner DATE: 11/5/80

CALVERT CLIFFS 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-318
2. Utility Contact: S. D. Merson (301) 234-5240
3. Licensed Thermal Power (MWt): 2700
4. Nameplate Rating (Gross MWe): 1012 x 0.9 = 911
5. Design Electrical Rating (Net MWe): 845
6. Maximum Dependable Capacity (Gross MWe): 860
7. Maximum Dependable Capacity (Net MWe): 825
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

ITEM 21 CALCULATED WITH A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	6,575.0	30,695.0
12. Hours Reactor Was Critical	671.8	6,272.1	26,217.0
13. Reactor Reserve Shutdown Hours	.0	22.5	435.6
14. Hours Generator On-Line	658.5	6,228.4	25,836.9
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,530,389	15,719,732	63,963,963
17. Gross Electrical Energy (MWH)	484,132	5,164,587	21,169,810
18. Net Electrical Energy (MWH)	458,792	4,931,836	20,188,856
19. Unit Service Factor	91.5	94.7	84.2
20. Unit Availability Factor	91.5	94.7	84.2
21. Unit Capacity Factor (MDC Net)	77.2	90.9	80.5
22. Unit Capacity Factor (DER Net)	75.4	88.8	79.7
23. Unit Forced Outage Rate	4.0	1.4	5.6

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
REFUELING, MAINT., INSPECT. & TMI MODS. - 1/2 3/2/81
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-06	09/07/80	S	24.0	H	5		XX	ZZZZZZ	FUEL CONSERVATION, CAPACITY AVAILABLE ON DEMAND.
80-07	09/12/80	S	34.0	B	1		XX	SUPORT	PLANNED SNUBBER INSPECTION.
80-08	09/14/80	F	27.5	H	1		XX	ZZZZZZ	TO COMPLY WITH TECHNICAL SPECIFICATIONS TO REPAIR ERRATIC LEVEL TRANSMITTER.
80-09	09/20/80	S	65.0	H	5		XX	ZZZZZZ	FUEL CONSERVATION, CAPACITY AVAILABLE ON DEMAND.
80-10	09/27/80	S	50.0	H	5		XX	ZZZZZZ	FUEL CONSERVATION, CAPACITY AVAILABLE ON DEMAND.

SUMMARY

CALVERT CLIFFS II OPERATED WITH THREE REDUCTIONS FOR FUEL CONSERVATION, AND TWO OUTAGES FOR SNUBBER INSPECTION AND LEVEL TRANSMITTER REPAIR.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

COOK 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Michigan/Berrien 11 miles S of Benton Harbor
2.	Electric Reliability Council	East Central Area Reliability Coordination Agreement
3.	Type of Reactor	PWR
4.	Power Level (MWT)	3250
5.	Design Electrical Rating (MWE Net)	1054
6.	Date of Initial Criticality	January 18, 1975
7.	Date of Commercial Operation	August 27, 1975
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10.	Licensee	AEP Corporation
11.	Corporate Address	Indiana & Michigan Electric Co. 2 Broadway New York, New York 10004
12.	Corporate Contact	John Dolan Senior Executive V. P. Eng.
13.	Architect/Engineer	AEP
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Indiana & Michigan Power Co.

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	D.C. Boyd
19.	IE Resident Inspector	E. Swanson (616) 465-6775
20.	Licensing Project Manager	S. Miner
21.	Docket Number	50-315
22.	License Number/Date of Issue	DPR-58 issued 10/25/74
* 23.	Public Document Room	Maude Preston Palenske Memorial Library 500 Market Street St. Joseph, Michigan 49085

INSPECTION STATUS

ENFORCEMENT STATUS

10 CFR 50, APPENDIX B, CRITERION II REQUIRES THE ESTABLISHMENT OF A PROGRAM FOR INDOCTRINATION AND TRAINING OF PERSONNEL PERFORMING ACTIVITIES AFFECTING QUALITY AS NECESSARY TO ASSURE THAT SUITABLE PROFICIENCY IS ACHIEVED AND MAINTAINED. POWER SYSTEMS, INC. QCP-12, "TRAINING AND CERTIFICATION," REQUIRES THE DEVELOPMENT OF A PROGRAM, OUTLINES PREPARED AND AIDS AVAILABLE; THE PROGRAM SHALL INCLUDE TRAINING RECORDS, MONTHLY SCHEDULES, ATTENDANCE RECORDS, AND COURSE OUTLINES MAINTAINED. ANSI-N45.2.6, SECTION 2.2, REQUIRES THAT EACH PERSON WHO VERIFIES CONFORMANCE OF WORK ACTIVITIES TO QUALITY REQUIREMENTS BE CERTIFIED BY HIS EMPLOYER AS BEING QUALIFIED TO PERFORM HIS ASSIGNED WORK. CONTRARY TO THE ABOVE, ON JUNE 18, 1980, DURING AN EXAMINATION OF POWER SYSTEMS ACTIVITIES AND PERSONNEL QUALIFICATIONS, THE INSPECTOR COULD FIND NO EVIDENCE OF A PROGRAM FOR INDOCTRINATION, TRAINING, OR CERTIFICATION OF QUALITY CONTROL INSPECTORS. NO CERTIFICATION RECORDS EXIST FOR FOUR INSPECTORS PERFORMING QUALITY INSPECTIONS.
(8010 INFRACTION)

VITAL AREA BARRIER WAS FOUND TO BE DEGRADED.
(8014 INFRACTION)

LICENSEE FAILED TO MAINTAIN AN ADEQUATE ISOLATION ZONE.
(8014 INFRACTION)

LICENSEE FAILED TO CONDUCT PROPER SCREENING IN ACCORDANCE WITH SECURITY PLAN COMMITMENTS.
(8014 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

Operating routinely.

*LAST IE SITE INSPECTION DATE: August 18-22, 1980

* INSPECTION REPORT NO: 80-14

SUMMARY

INSPECTION SUMMARY

* Inspection on August 18-22, (80-14): Included a review of the Security Plan; Security Program Audit; Security Organization; Protected Area Physical Barriers; Vital Area Physical Barriers; Lighting; Access Control (Identification, Authorization, and Badging); Access Control (Search); Access Control (Escorting); Access Control (Vital Areas); Alarm Stations; Detection Aids; Assessment Aids; Communications; Testing and Maintenance; Compensatory Measures; Power Supply; Response; Locks, Keys and Combinations; Records and Reports; Licensee Action of Previous Inspection Findings; and Independent Inspection Effort. The inspection involved 132 inspector hours onsite by three NRC inspectors. The inspection began during the day shift; 30 inspector hours were accomplished during off-shift periods. The licensee was found to be in compliance with NRC requirements within the areas examined during the inspection, except for the following three items: Access Control (Identification, Authorization, and Badging) - The licensee failed to conduct proper personnel screening in accordance with security plan commitments; Physical Barriers (Vital Areas) - A vital area barrier was found to be degraded; Physical Barriers (Protected Area) - The licensee failed to maintain an adequate isolation zone. The items of noncompliance from the previous security inspection (IE Inspection Report Nos. 50-315/80-05; 50-316/80-04), with the exception of one item were closed. One unresolved item identified in a previous security inspection remains open.

REPORTS RECEIVED FROM LICENSEE

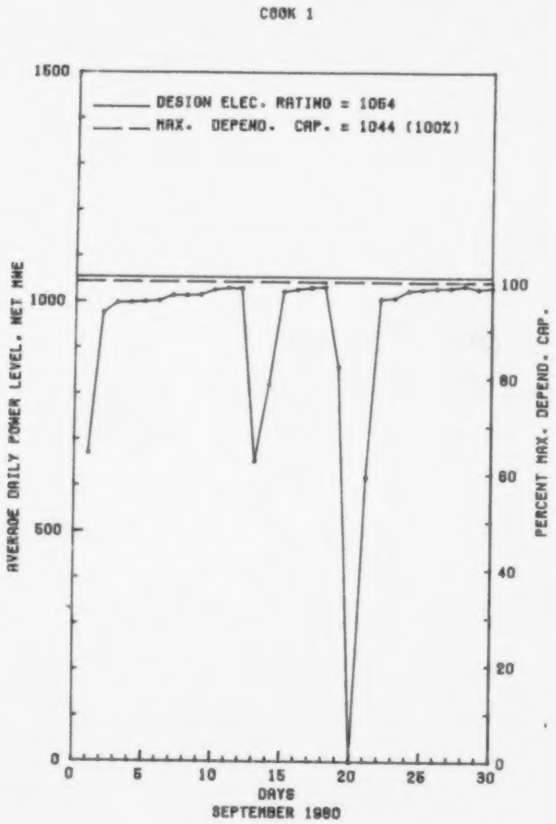
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-17/ 03L-0	08/04/80	09/03/80	During low power physics testing following refueling the boron injection tank, after a routine sampling and analysis was discovered to have a concentration of 19731 PPM.
80-19/ 03L-0	08/11/80	09/10/80	While performing surveillance to meet action required of T.S. because of an inoperable diesel generator, the surveillance required of T.S. was not performed.
80-20/ 03L-0	08/31/80	09/24/80	During normal plant operation a reactor trip train A safety injection and steam line isolation occurred as a result of the loss of a 120 volt A.C. Vital Bus.
80-21/ 03L-0	08/17/80	09/12/80	Two intermediate deck doors were found frozen and declared inoperable per T.S.

REVIEWED BY OMPA LOEB: R E Bobe **DATE:** 11/4/80

REVIEWED BY NRR DL: Sydney Miner **DATE:** 11/5/80

COOK 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-315
2. Utility Contact: W. T. Gillett (616) 465-5901
3. Licensed Thermal Power (MWt): 3250
4. Nameplate Rating (Gross MWe): 1280 x 0.9 = 1152
5. Design Electrical Rating (Net MWe): 1054
6. Maximum Dependable Capacity (Gross MWe): 1080
7. Maximum Dependable Capacity (Net MWe): 1044
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	6,575.0	50,399.0
12. Hours Reactor Was Critical	696.0	4,548.3	37,510.0
13. Reactor Reserve Shutdown Hours	.0	.0	463.0
14. Hours Generator On-Line	691.7	4,466.0	36,549.2
15. Unit Reserve Shutdown Hours	.0	.0	321.0
16. Gross Thermal Energy (MWH)	2,100,551	13,721,043	102,787,348
17. Gross Electrical Energy (MWH)	698,140	4,545,040	33,707,380
18. Net Electrical Energy (MWH)	672,667	4,381,662	32,378,976
19. Unit Service Factor	96.1	67.9	75.7
20. Unit Availability Factor	96.1	67.9	75.7
21. Unit Capacity Factor (MDC Net)	89.5	63.8	66.5
22. Unit Capacity Factor (DER Net)	88.6	63.2	62.2
23. Unit Forced Outage Rate	3.9	9.8	7.3

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
166	09/13/80	S	0	B	5		ZZ	ZZZZZZ	REACTOR POWER REDUCED TO 55% TO REMOVE THE WEST MAIN FEED-PUMP TURBINE FROM SERVICE TO REPAIR OIL LEAKS IN THE FEED-PUMP TURBINE LUBE-OIL TANK. REACTOR POWER RETURNED TO 98% 09/15/80.
167	09/19/80	F	28.3	B	1	80-023	IB	INSTRU	UNIT REMOVED FROM SERVICE TO REPAIR SUSPECTED LEAK IN SENSING LINE FOR NO. 2 STEAM GENERATOR NARROW RANGE LEVEL CHANNEL BLP-122. LEAK FOUND TO BE PINHOLE LEAK IN A SOCKET WELD DOWNSTREAM OF ROOT VALVE IN THE SENSING LINE. UNIT RETURNED TO SERVICE 09/21/80, AND 97% REACTOR POWER REACHED 09/22/80.

SUMMARY

COOK 1 OPERATED NORMALLY WITH ONE REDUCTION FOR FEEDPUMP OIL LEAKS ON THE 13TH AND AN OUTAGE ON THE 19TH TO REPAIR SENSING LINE LEAK.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

COOK 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Michigan/Berrien 11 miles S of Benton Harbor
2.	Electric Reliability Council	East Central Area Reliability Coordination Agreement
3.	Type of Reactor	PWR
4.	Power Level (MWT)	3391
5.	Design Electrical Rating (MWE Net)	1100
6.	Date of Initial Criticality	March 10, 1978
7.	Date of Commercial Operation	July 1, 1978
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10.	Licensee	AEP Corporation Indiana & Michigan Electric Co.
11.	Corporate Address	2 Broadway New York, New York 10004
12.	Corporate Contact	John Dolan, VP/Chief Engineer (212) 422 - 4800
13.	Architect/Engineer	American Elec. Power Service Corp.
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Indiana & Michigan Power Co.

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	D.C. Boyd
19.	IE Resident Inspector	E. Swanson (616) 465-6775
20.	Licensing Project Manager	S. Miner
21.	Docket Number	50-316
22.	License Number/Date of Issue	DPR-74, 12/23/77
23.	Public Document Room	Maude Preston Palenske Memorial Library 500 Market Street St. Joseph, Michigan 49085

INSPECTION STATUS

ENFORCEMENT STATUS

LICENSEE FAILED TO MAINTAIN AN ADEQUATE ISOLATION ZONE.
(8013 INFRACTION)

VITAL AREA BARRIER WAS FOUND TO BE DEGRADED.
(8013 INFRACTION)

LICENSEE FAILED TO CONDUCT PROPER SCREENING IN ACCORDANCE WITH SECURITY PLAN COMMITMENTS.
(8013 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

Operating routinely.

+LAST IE SITE INSPECTION DATE: August 18-22, 1980

+ INSPECTION REPORT NO: 80-13

SUMMARY

INSPECTION SUMMARY

+ Inspection on August 18-22, (80-13): Included a review of the Security Plan; Security Program Audit; Security Organization; Protected Area Physical Barriers; Vital Area Physical Barriers; Lighting; Access Control (Identification, Authorization, and Badging); Access Control (Search); Access Control (Escorting); Access Control (Vital Areas); Alarm Stations; Detection Aids; Assessment Aids; Communications; Testing and Maintenance; Compensatory Measures; Power Supply; Response; Locks, Keys and Combinations; Records and Reports; Licensee Action of Previous Inspection Findings; and Independent Inspection Effort. The inspection involved 132 inspector hours onsite by three NRC inspectors. The inspection began during the day shift; 30 inspector hours were accomplished during off-shift periods. The licensee was found to be in compliance with NRC requirements within the areas examined during the inspection, except for the following three items: Access Control (Identification, Authorization, and Badging) - The licensee failed to conduct proper personnel screening in accordance with security plan commitments; Physical Barriers (Vital Areas) - A vital area barrier was found to be degraded; Physical Barriers (Protected Area) - The licensee failed to maintain an adequate isolation zone. The items of noncompliance from the previous security inspection (IE Inspection Report Nos. 50-315/80-05; 50-316/80-04), with the exception of one item were closed. One unresolved item identified in a previous security inspection remains open.

REPORTS RECEIVED FROM LICENSEE

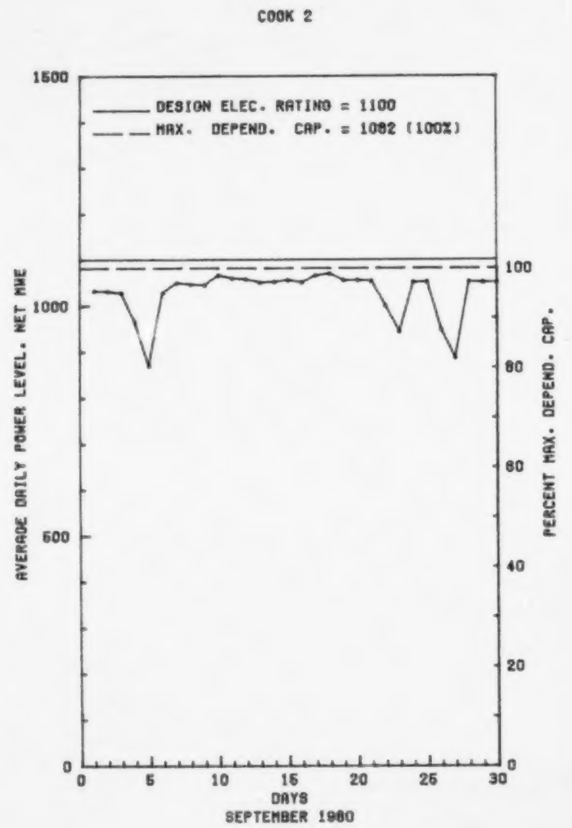
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-30/03L-0	08/14/80	09/12/80	During normal operation, Rad Monitor R-11 alarmed, because the R-11 and R-12 sample pump had seized.

REVIEWED BY OMPA LOEB: PE Kobe **DATE:** 11/4/80

REVIEWED BY NRR DL: **DATE:** 11/5/80

COOK 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-316
2. Utility Contact: W. T. Gillett (616) 465-5901
3. Licensed Thermal Power (MWt): 3391
4. Nameplate Rating (Gross MWe): 1333 x 0.85 = 1133
5. Design Electrical Rating (Net MWe): 1100
6. Maximum Dependable Capacity (Gross MWe): 1118
7. Maximum Dependable Capacity (Net MWe): 1082
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	6,575.0	24,095.0
12. Hours Reactor Was Critical	720.0	5,731.4	16,811.3
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	720.0	5,639.4	16,141.3
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	2,393,783	18,492,353	50,716,733
17. Gross Electrical Energy (MWH)	767,140	6,081,330	16,155,740
18. Net Electrical Energy (MWH)	739,915	5,789,125	15,556,537
19. Unit Service Factor	100.0	85.8	75.7
20. Unit Availability Factor	100.0	85.8	75.7
21. Unit Capacity Factor (MDC Net)	95.0	81.4	68.6
22. Unit Capacity Factor (DER Net)	93.4	80.0	63.8
23. Unit Forced Outage Rate	.0	1.6	5.1

24. Shutdowns Scheduled Over Next 6 Months (Type,Date,and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
83	09/04/80	F	.0	A	5		ZZ	ZZZZZZ	REACTOR POWER REDUCED TO 54% TO REMOVE EAST MAIN FEEDPUMP TURBINE FROM SERVICE TO CHECK FEEDPUMP TURBINE CONDENSER FOR TUBE LEAKS. TWO TUBES WERE PLUGGED. REACTOR POWER RETURNED TO 100% 09/05/80.
84	09/22/80	F	.0	A	5		ZZ	ZZZZZZ	REACTOR POWER REDUCED TO 53% TO REMOVE THE EAST MAIN FEEDPUMP TURBINE FROM SERVICE TO CHECK FEEDPUMP TURBINE CONDENSER FOR TUBE LEAKS. SIX LEAKY TUBES WERE PLUGGED. REACTOR POWER RETURNED TO 100% 09/23/80.
85	09/26/80	F	.0	A	5		ZZ	ZZZZZZ	REACTOR POWER REDUCED TO 60% TO REMOVE EAST MAIN FEEDPUMP TURBINE FROM SERVICE TO CHECK FEEDPUMP TURBINE CONDENSER FOR TUBE LEAKS. ONE TUBE WAS PLUGGED. REACTOR POWER RETURNED TO 100% 09/27/80.

SUMMARY

COOK 2 OPERATED AT NEAR FULL POWER FOR THE MONTH, WITH THREE REDUCTIONS FOR EQUIPMENT FAILURE ON THE 4TH, 22ND AND 26TH.

LEGEND

[1]Type:
F--Forced
S--Scheduled

[2]Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3]Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4]Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5]Exhibit H:
Same source.

COOPER STATION

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Nebraska/Nemaha 23 miles S of Nebraska City
2.	Electric Reliability Council	Mid-Continent Area Reliability Coordination Agreement
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2381
5.	Design Electrical Rating (MWE Net)	778
6.	Date of Initial Criticality	February 21, 1974
7.	Date of Commercial Operation	July 1, 1974 (50% rating)
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Missouri River

Utility & Contractor Information

10.	Licensee	Nebraska Public Power District
11.	Corporate Address	P. O. Box 499 Columbus, Nebraska 68601
12.	Corporate Contact	J. M. Pilant, Director Licensing and Quality Assurance
13.	Architect/Engineer	Burns and Roe, Inc.
14.	Nuclear Steam Supply System	GE
15.	Constructor	Burns and Roe, Inc.

Regulatory Information

16.	IE Region responsible	IV
17.	IE Branch Chief	G. L. Madsen
18.	IE Section Chief	T. F. Westerman
19.	IE Resident Inspector	R. G. Spangler
20.	Licensing Project Manager	V. L. Rooney
21.	Docket Number	50-298
22.	License Number/Date of Issue	DPR-46, 1/18/74
23.	Public Document Room	Auburn Public Library 118 15th Street Auburn, Nebraska 68305

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO 10CFR20.103 AND REG GUIDE 8.15, A PAINTER PERFORMING GRINDING IN THE TORUS WAS PROVIDED WITH A RESPIRATORY PROTECTIVE DEVICE WHICH DID NOT PROVIDE A PROTECTION FACTOR GREATER THAN THE MULTIPLE BY WHICH THE PEAK CONCENTRATION OF RADIOACTIVE MATERIAL EXCEEDED THE VALUES SPECIFIED IN TABLE 1 COL. 1 OF APPENDIX B TO 10CFR 20.
(8007 INFRACTION)

CONTRARY TO TECH SPEC 6.3.5, DURING THE PERIOD MAY 5-16, 1980 INDIVIDUALS WERE OBSERVED ENTERING THE REACTOR DRYWELL, A HIGH RADIATION AREA, WITHOUT NOTIFYING THE SHIFT SUPERVISOR AND WITHOUT A DOSE RATE INSTRUMENT AS REQUIRED.
(8007 INFRACTION)

CONTRARY TO TECH SPEC 6.1.4 WHICH STATES THAT THE MINIMUM QUALIFICATIONS OF PLANT PERSONNEL SHALL MEET THE REQUIREMENTS DESCRIBED IN ANSI N18.1-1971, TWO HEALTH PHYSICS TECHNICIANS, HAVING APPROXIMATELY SIX AND TEN MONTHS OF WORKING EXERIENCE, WERE APPOINTED TO RESPONSIBLE POSITIONS PRIOR TO ATTAINING TWO YEARS OF EXPERIENCE REQUIRED BY ANSI N18.1-1971, SECT 4.5.2.
(8007 INFRACTION)

10 CFR 55 APPENDIX A, REQUALIFICATION PROGRAM REQUIREMENTS, REQUIRES THAT THE REQUALIFICATION PROGRAM INCLUDE PREPLANNED LECTURES ON A REGULAR AND CONTINUING BASIS THROUGHOUT THE LICENSE PERIOD IN THOSE AREAS WHERE ANNUAL OPERATOR AND SENIOR OPERATOR WRITTEN EXAMINATIONS INDICATE THAT SCOPE AND DEPTH OF COVERAGE IS NEEDED. THIS IS AMPLIFIED IN COOPER NUCLEAR STATION'S REQUALIFICATION PROGRAM APPROVED BY NRR CN AUG 18, 1976 WHICH STATES, "WITH THE EXCEPTION OF SPECIAL ACTIVITY PERIODS, SUCH AS REFUELING OUTAGES, THE LECTURE SERIES WILL BE SPREAD REASONABLY EVENLY THROUGHOUT EACH YEAR." 10 CFR 55, APPENDIX A, REQUALIFICATION PROGRAM REQUIREMENTS, ALSO REQUIRES THE REQUALIFICATION PROGRAM INCLUDE WRITTEN EXAMINATIONS WHICH DETERMINE LICENSED OPERATOR'S AND SENIOR OPERATOR'S KNOWLEDGE OF SUBJECTS COVERED IN THE REQUALIFICATION PROGRAM AND PROVIDE A BASIS FOR EVALUATING THEIR KNOWLEDGE OF ABNORMAL AND EMERGENCY PROCEDURES. THE LICENSEE'S APPROVED REQUALIFICATION PROGRAM STATES, "WRITTEN EXAMINATIONS SHALL BE GIVEN COVERING MATERIAL PRESENTED IN THE REQUALIFICATION PROGRAM LECTURE SERIES." CONTRARY TO THE ABOVE: 1. THE LICENSEE HAD NOT INITIATED A REQUALIFICATION PROGRAM LECTURE SERIES FOR THE 1980 TRAINING CYCLE BEGINNING IN FEB 1980. 2. REQUALIFICATION QUIZ NO. 22, ADMINISTERED IN NOV 1979 HAD NOT BEEN GRADED TO DETERMINE THE KNOWLEDGE THAT LICENSED OPERATORS AND SENIOR OPERATORS HAD IN THE SUBJECTS COVERED IN THE REQUALIFICATION PROGRAM LECTURE SERIES.
(8013 INFRACTION)

THE LICENSEE'S APPROVED OPERATOR REQUALIFICATION PROGRAM STATES THAT "A PLANNED LECTURE SERIES SHALL BE PRESENTED COVERING, AS A MINIMUM, THOSE AREAS WHERE ANNUAL EXAMINATIONS INDICATE THE NEED FOR ADDITIONAL TRAINING IN THE FOLLOWING SUBJECTS: "1. THEORY AND PRINCIPLES OF OPERATION. "10. STATION QA PROGRAM AS RELATED TO STATION OPERATIONS." IN DEVIATION FROM THE ABOVE, THE ANNUAL EXAMINATION ADMINISTERED IN FEB-APR 1980, DID NOT ADDRESS THE SUBJECT OF "STATION QA PROGRAM AS RELATED TO STATION OPERATIONS." (298/80-13-04).
(8013 DEVIATION)

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-33 03L-0	7/28/80	8/8/80	Inboard DW isolation valve failed to close when tested

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

FACILITY ITEMS (PLANS AND PROCEDURES):

MANAGERIAL ITEMS:

PLANT STATUS:

OPERATING AT RATED THERMAL POWER.

LAST IE SITE INSPECTION DATE: September 8 - 11, 1980 +

INSPECTION REPORT NO: 50-298/80-13 +

SUMMARY

INSPECTION SUMMARY

+ Inspection on September 8-11, 1980 (80-13): Routine, unannounced inspection of review of plant operations, requalification training, followup on LER's and followup on IE Bulletins and Circulars. Within the four areas inspected no items of noncompliance or deviations were identified in three of the areas. Within the area of requalification training one item of noncompliance (infraction - failure to fully implement the requirements of Appendix A to 10 CFR 55 and one deviation (failure to fully meet a committment made in the operator requalification program were identified).

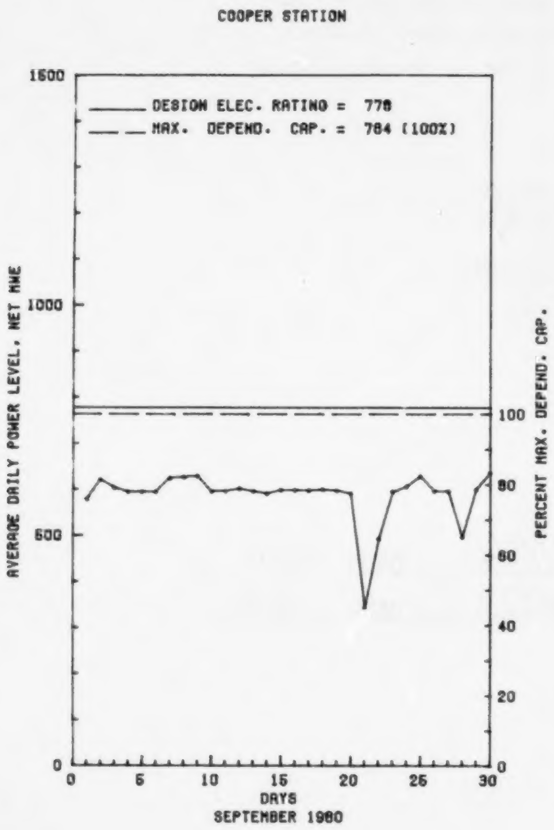
REVIEWED BY OMPA LOEB: ____________ **DATE:** ____________

REVIEWED BY NRR DL: ____________ **DATE:** 11/6/80

C-10

COOPER STATION

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-298
2. Utility Contact: P. J. Borer (402) 825-3811
3. Licensed Thermal Power (MWt): 2381
4. Nameplate Rating (Gross MWe): 983 x 0.85 = 836
5. Design Electrical Rating (Net MWe): 778
6. Maximum Dependable Capacity (Gross MWe): 787
7. Maximum Dependable Capacity (Net MWe): 764
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): 640
10. Reasons for Restrictions, If Any: TEMPORARY TURBINE MODIFICATION

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	6,575.0	54,816.0
12. Hours Reactor Was Critical	720.0	4,264.7	45,502.0
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	720.0	4,170.4	44,646.0
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,580,424	8,959,752	85,142,478
17. Gross Electrical Energy (MWH)	440,470	2,638,393	27,177,241
18. Net Electrical Energy (MWH)	422,898	2,535,047	26,193,324
19. Unit Service Factor	100.0	63.4	81.4
20. Unit Availability Factor	100.0	63.4	81.4
21. Unit Capacity Factor (MDC Net)	76.9	50.5	62.5
22. Unit Capacity Factor (DER Net)	75.5	49.6	61.4
23. Unit Forced Outage Rate	.0	2.1	4.4

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-11	09/21/80	S	.0	B	5				REDUCED POWER TO ALLOW EXCHANGE OF THE CONTROL ROD SEQUENCES.

SUMMARY

COOPER STATION OPERATED NORMALLY FOR THE REPORT PERIOD WITH NO OUTAGES AND ONE REDUCTION FOR CONTROL ROD ADJUSTMENTS.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

CRYSTAL RIVER 3

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Florida/Citrus 7 miles NW of Crystal River
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2452
5.	Design Electrical Rating (MWE Net)	825
6.	Date of Initial Criticality	January 14, 1977
7.	Date of Commercial Operation	March 13, 1977
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Gulf of Mexico

Utility & Contractor Information

10.	Licensee	Florida Power Corporation
11.	Corporate Address	3201 34th Street, South St. Petersburg, Florida 33733
12.	Corporate Contact	J. Hancock
13.	Architect/Engineer	Gilbert Associates, Inc.
14.	Nuclear Steam Supply System	B&W
15.	Constructor	J. A. Jones Construction Co.

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. Lewis
18.	IE Section Chief	R. Martin
19.	IE Resident Inspector	T. F. Stetka (904) 795-7677
20.	Licensing Project Manager	M. B. Fairtile
21.	Docket Number	50-302
22.	License Number/Date of Issue	DPR-72, 12/3/76
23.	Public Document Room	Crystal River Public Library 668 N.W. First Crystal River, Florida 32639

INSPECTION STATUS

ENFORCEMENT STATUS

AS REQUIRED IN PART BY 10 CFR 20.102.B, "BEFORE PERMITTING ANY INDIVIDUAL IN A RESTRICTED AREA TO RECEIVE AN OCCUPATIONAL RADIATION DOSE IN EXCESS OF THE STANDARDS SPECIFIED IN 20.101(A), EACH LICENSEE SHALL: OBTAIN A CERTIFICATE ON FORM NRC-4, OR ON A CLEAR AND LEGIBLE RECORD CONTAINING ALL THE INFORMATION REQUIRED IN THAT FORM, SIGNED BY THE INDIVIDUAL SHOWING EACH PERIOD OF TIME AFTER THE INDIVIDUAL ATTAINED THE AGE OF 18 IN WHICH THE INDIVIDUAL RECEIVED AN OCCUPATIONAL DOSE OF RADIATION. CONTRARY TO THE ABOVE, TWO WORKERS WITH INCOMPLETE FORM NRC-4'S WERE PERMITTED BY DOSIMETRY CONTROL PRINTOUT TO EXCEED 1.25 REM PER QUARTER AND ONE OF THE WORKERS RECEIVED AN INDICATED 1,274 MILLIREM.
(8025 INFRACTION)

AS REQUIRED BY 10 CFR 20.103.C, "WHEN RESPIRATORY PROTECTIVE EQUIPMENT IS USED TO LIMIT THE INHALATION OF AIRBORNE RADIOACTIVE MATERIAL, THE LICENSEE MAY MAKE ALLOWANCES FOR SUCH USE IN ESTIMATING EXPOSURES OF INDIVIDUALS TO SUCH MATERIALS PROVIDED THAT SUCH EQUIPMENT IS USED AS STIPULATED IN REGULATORY GUIDE 8.15, "ACCEPTABLE PROGRAMS FOR RESPIRATORY PROTECTION. CONTRARY TO THE ABOVE, ON JUNE 21, 1980, (A) THE ISSUANCE RECORDS FOR RESPIRATORS INDICATED AT LEAST THREE INDIVIDUALS WHO HAD NOT BEEN MEDICALLY EXAMINED IN THE PAST 15 MONTHS AND ONE INDIVIDUAL WHO HAD NOT BEEN MEDICALLY EXAMINED FOR RESPIRATOR USE IN THE PAST TWO YEARS, AND, (B) THE RESPIRATORY PROGRAM DOES NOT REQUIRE TERMINATION WHOLE BODY COUNTS FOR ANYONE, NOR DOES IT REQUIRE COMMENCEMENT OR TERMINATION WHOLE BODY COUNT FOR VISITORS.
(8025 INFRACTION)

AS REQUIRED BY 10 CFR 20.203.B, "EACH RADIATION AREA SHALL BE CONSPICUOUSLY POSTED WITH A SIGN OR SIGNS BEARING THE RADIATION CAUTION SYMBOL AND THE WORDS: CAUTION-RADIATION AREA. CONTRARY TO THE ABOVE, RADIATION AREAS WERE NOT POSTED FOR: (A) ON JUNE 23, 1980, AN OPEN LSA BOX IN THE COMPACTOR ROOM READ 30 MILLIREM PER HOUR AT 18 INCHES; (B) ON JUNE 25, 1980, A GENERAL AREA OF 25 MILLIREM PER HOUR WAS MEASURED OUTSIDE THE COMPACTOR ROOM AND THE ROOM WAS UNMARKED; AND (C) ON JUNE 25, 1980, THE SAFEGUARDS AREA PIPE GALLERY AT THE 95 FOOT ELEVATION WAS UNPOSTED AND MEASURED 15 MILLIREM PER HOUR.
(8025 INFRACTION)

AS REQUIRED BY 10 CFR 20.203.C.1, EACH HIGH RADIATION AREA SHALL BE CONSPICUOUSLY POSTED WITH A SIGN OR SIGNS BEARING THE RADIATION CAUTION SYMBOL AND THE WORDS: "CAUTION - HIGH RADIATION AREA". CONTRARY TO THE ABOVE, ON JUNE 18, 1980, A HIGH RADIATION AREA WAS NOT CONSPICUOUSLY POSTED IN THE CONTAINMENT NEAR THE REACTOR VESSEL HEAD STORAGE STAND. LEVELS OF 2.2 REM PER HOUR ON CONTACT AND 150 MILLIREM PER HOUR WERE MEASURED ON A VACUUM CLEANER IN THE AREA AND 100 MILLIREM PER HOUR AT 18 INCHES FROM A SHIELDED MOTOR TUBE HOUSING. WORKERS IN THE AREA WERE NOT AWARE OF THESE DOSE RATE LEVELS.
(8025 INFRACTION)

AS REQUIRED BY 10 CFR 20.201, "EACH LICENSEE SHALL MAKE OR CAUSE TO BE MADE SUCH SURVEYS AS MAY BE NECESSARY FOR HIM TO COMPLY WITH REGULATIONS IN THIS PART." 10 CFR 20.103.C. STATES "THE LICENSEE SHALL USE SUITABLE MEASUREMENTS OF CONCENTRATIONS OF RADIOACTIVE MATERIALS IN AIR FOR DETECTING AND EVALUATING AIRBORNE RADIOACTIVITY IN RESTRICTED AREAS...". CONTRARY TO THE ABOVE, ON JUNE 19, 1980, THREE PEOPLE WORKING IN THE VICINITY OF THE REACTOR VESSEL HEAD BOLTS WERE WEARING RESPIRATORY PROTECTION AND NO AIRBORNE RADIOACTIVITY SURVEY WAS TAKEN IN THE BREATHING ZONE.
(8025 INFRACTION)

AS REQUIRED BY TECHNICAL SPECIFICATION 6.4.1, A RETRAINING PROGRAM FOR THE FACILITY STAFF SHALL BE MAINTAINED AND SHALL MEET OR EXCEED THE REQUIREMENTS AND RECOMMENDATIONS OF SECTION 5.5 OF ANSI N18.1-1971. SECTION 5.5 OF ANSI N18.1 STATES THAT A TRAINING PROGRAM SHALL BE ESTABLISHED WHICH MAINTAINS THE PROFICIENCY OF THE OPERATING ORGANIZATION. CONTRARY TO THE ABOVE, THE LICENSEE DID NOT HAVE A FORMAL RETRAINING PROGRAM WHICH MAINTAINED THE PROFICIENCY OF THE HEALTH PHYSICS TECHNICIANS.
(8025 INFRACTION)

AS REQUIRED BY TECHNICAL SPECIFICATION 6.12.1 A ENTRANCE TO A HIGH RADIATION AREA "SHALL BE CONTROLLED BY ISSUANCE OF A RADIATION WORK PERMIT AND ANY INDIVIDUAL OR GROUP OF INDIVIDUALS PERMITTED TO ENTER SUCH AREAS SHALL BE PROVIDED WITH A RADIATION MONITORING DEVICE WHICH CONTINUOUSLY INDICATES THE RADIATION DOSE RATE IN THE AREA." CONTRARY TO THE ABOVE, ON JUNE 16, 1980, TWO WORKERS ENTERED A HIGH RADIATION AREA IN THE VICINITY OF THE REACTOR VESSEL FLANGE DURING THE LOWERING OF THE VESSEL HEAD WITHOUT A SURVEY INSTRUMENT, AND ON JUNE 20, 1980, ONE WORKER ENTERED A HIGH RADIATION AREA IN THE "BACKYARD" NEAR THE CHEM-NUCLEAR TRAILER WITHOUT A DOSE RATE SURVEY INSTRUMENT IN THE PRESENCE OF A HEALTH PHYSICS TECHNICIAN.
(8025 INFRACTION)

AS REQUIRED BY TECHNICAL SPECIFICATION 6.11 "PROCEDURES FOR PERSONNEL RADIATION PROTECTION SHALL BE PREPARED CONSISTENT WITH THE REQUIREMENTS OF 10 CFR PART 20 AND SHALL BE APPROVED, MAINTAINED AND ADHERED TO FOR ALL OPERATIONS INVOLVING PERSONNEL RADIATION EXPOSURE." PROCEDURE RP-213, "POCKET DOSIMETER FUNCTIONAL CHECK," STATES IN SECTION 2.1.1 THAT "THE VARIOUS DOSIMETERS WILL BE CALIBRATION CHECKED ON A ROTATIONAL BASIS AT SIX MONTH INTERVALS OR AT SUCH TIMES AS THERE IS REASONABLE DOUBT AS TO THE AUTHENTICITY OF THE DOSIMETER'S PERFORMANCE. THE CYCLING FOR CALIBRATION PURPOSES SHALL BE SUCH THAT AN OPERATING SUPPLY OF DOSIMETERS IS READILY AVAILABLE AT ALL TIMES." CONTRARY TO THE ABOVE, ON JUNE 25, 1980, THE CALIBRATION PROGRAM AS PRACTICED ACTUALLY CALIBRATES ANNUALLY AND A REVIEW OF RECORDS INDICATED POCKET DOSIMETERS 007 AND 581 WERE LAST CALIBRATED ON FEBRUARY 8, 1980.
(8025 INFRACTION)

CONTINUED ON PAGE C-13A.

REPORTS RECEIVED FROM LICENSEE

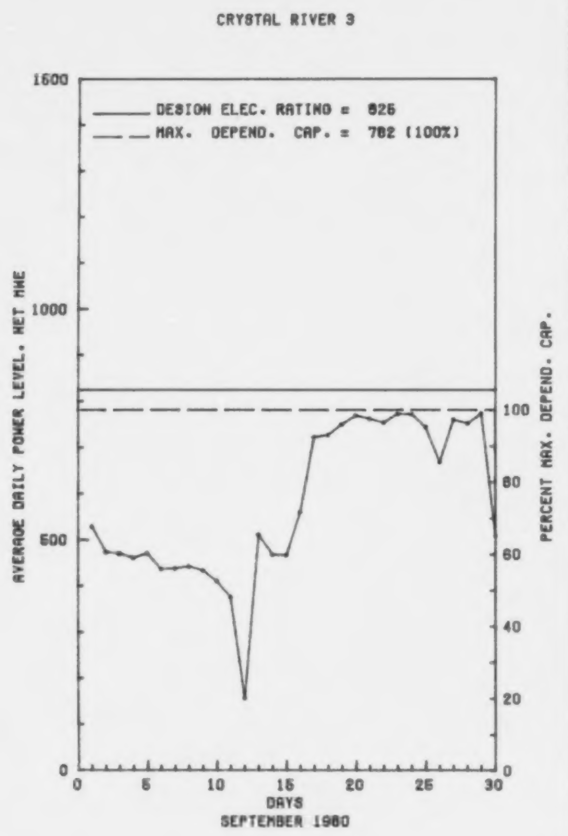
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
79-102/ 03L-0	10/23/79	11/15/79	INOPERABLE CORE FLOOD TANK SAMPLE VALVE CFV-12.
80-027/ 01T	08/03/80	08/29/80	FAILURE OF CORE FLOOD TANK SAMPLE ISOLATION VALVE TO CLOSE.
80-030/ 03L-0	07/31/80	08/22/80	EMERGENCY DIESEL GENERATOR TURBOCHARGER DISCHARGE DUCT SEPARATED DUE TO VIBRATION.
80-032/ 03L-0	08/05/80	08/29/80	DURING LOAD TEST OF DIESEL GENERATOR THE GENERATOR TRIPPED OFF.
80-035/ 03L-0	08/16/80	09/11/80	STARTUP FEEDWATER FLOW INDICATORS WERE INACCURATE DUE TO INSTRUMENT DRIFT.

REVIEWED BY OMPA LOEB: PE Bole **DATE:** 11/4/80

REVIEWED BY NRR DL: PB Erickson **DATE:** 11/6/80

CRYSTAL RIVER 3

AVERAGE POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-302
2. Utility Contact: M.W. CULVER (904) 795-6486
3. Licensed Thermal Power (MWt): 2452
4. Nameplate Rating (Gross MWe): 989 x 0.9 = 890
5. Design Electrical Rating (Net MWe): 825
6. Maximum Dependable Capacity (Gross MWe): 821
7. Maximum Dependable Capacity (Net MWe): 782
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	6,575.0	31,151.0
12. Hours Reactor Was Critical	705.9	2,544.5	17,694.7
13. Reactor Reserve Shutdown Hours	8.1	8.1	1,125.0
14. Hours Generator On-Line	699.8	2,481.4	17,181.7
15. Unit Reserve Shutdown hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,357,454	5,339,165	37,656,076
17. Gross Electrical Energy (MWH)	443,395	1,815,739	12,787,507
18. Net Electrical Energy (MWH)	416,862	1,721,992	12,113,632
19. Unit Service Factor	97.2	37.7	55.2
20. Unit Availability Factor	97.2	37.7	55.2
21. Unit Capacity Factor (MDC Net)	74.0	33.5	49.7
22. Unit Capacity Factor (DER Net)	70.2	31.7	47.1
23. Unit Forced Outage Rate	.9	11.4	30.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-15	09/11/80	S	13.9	B	1		ZZ	ZZZZZZ	MINI MAINTENANCE OUTAGE - CLEANED CONDENSER WATERBOXES, PUT OIL IN RCP'S, REPLACED TWO (2) PI TUBES.
80-16	09/30/80	F	6.3	G	3		IA	INSTRU	DURING PERFORMANCE OF A SURVEILLANCE PROCEDURE ON RPS CHANNEL "A", A CLIP LEAD SLIPPED OFF AND SHORTED THE DC POWER SUPPLY. THIS CAUSED THE ICS TO SEE NO RC FLOW, AND SO IT RAN BACK FW. THE REACTOR TRIPPED ON HIGH RC PRESSURE.

SUMMARY

CRYSTAL RIVER 3 OPERATED NORMALLY FOR THE REPORT PERIOD, WITH AN OUTAGE ON THE 11TH FOR MAINTENANCE, AND ANOTHER ON THE 30TH DUE TO OPERATIONAL ERROR.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

AS REQUIRED BY TECHNICAL SPECIFICATION 6.11, "PROCEDURES FOR PERSONNEL RADIATION PROTECTION SHALL BE PREPARED CONSISTENT WITH THE REQUIREMENTS OF 10 CFR PART 20 AND SHALL BE APPROVED, MAINTAINED AND ADHERED TO FOR ALL OPERATIONS INVOLVING PERSONNEL RADIATION EXPOSURE." RADIOLOGICAL CONTROL PROCEDURE RP-102, REVISION 9, ENTITLED "RESPIRATORY EQUIPMENT MANUAL STATES: (1) IN SECTION 4, THAT "THE TRAINING PROGRAM SHALL, AS A MINIMUM, COVER THE FOLLOWING: DISCUSSION OF THE APPLICATION OF VARIOUS CARTRIDGES AND CANISTERS AVAILABLE FOR AIR-PURIFYING RESPIRATORS...(9 SEPARATE ITEMS). CONTRARY TO THE ABOVE, (1) ON JUNE 16, 1980, THE TRAINING PROGRAM DID NOT INCLUDE DISCUSSIONS OF THE TYPES OF CANISTERS AVAILABLE, FIELD TRAINING FOR EMERGENCIES, NOR FAMILIARIZATION WITH PLANT DEVICES (9 SEPARATE ITEMS).
(8025 INFRACTION)

CONTRARY TO 10CFR50 APPENDIX B CRITERION III, THE LICENSEE'S ACCEPTATED QA PROGRAM SECTION 1.7.6.1.5, REGULATORY GUIDE 1.64 AND ANSI 45-2-11-1974 DESIGN INPUTS WERE NOT CORRECTLY TRANSLATED INTO DRAWINGS FOR MAR 79-11-82G IN THAT WIRES TB 14-11-624 AND TB 14-12-624 WERE SPECIFIED TO BE CORRECTED IN RELAY RACK 3, RELAY BA, TERMINALS 9 AND 10 RESPECTIVELY. THESE TERMINALS DO NOT EXIST.
(8030 INFRACTION)

CONTRARY TO 10CFR50, APPENDIX B, CRITERION XVII, THE ACCEPTED QA PROGRAM (FSAR), SECTION 1.7.6.7.1.Q.2, AND QAP-2, PREPARATION AND CONTROL OF ADMINISTRATION PROCEDURES, REVISION 3 DATED 6/79 SUFFICIENT RECORDS WERE NOT MAINTAINED IN THAT THERE WERE NO RECORDS TO DOCUMENT THE RESULTS OF THE ANNUAL REVIEW OF QUALITY PROCEDURES.
(8030 DEFICIENCY)

CONTRARY TO T.S. 6.8.1 AND AI 1000, GOOD HOUSEKEEPING, REVISION 8 DATED 4/80 SECTION 8.0 PROCEDURES WERE NOT IMPLEMENTED BY THE BUILDING SERVICES SUPERINTENDENT TO CORRECT HOUSEKEEPING DEFICIENCIES FOR ITEMS DISCOVERED DURING INSPECTIONS ON 5/26-6/1, 6/9-14 AND 7/10/80.
(8030 INFRACTION)

INSPECTION STATUS

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

Routine operation.

- LAST IE SITE INSPECTION DATE: SEPTEMBER 29 - OCTOBER 1, 1980

- INSPECTION REPORT NO: 50-302/80-36

SUMMARY

INSPECTION SUMMARY

- INSPECTION JUNE 2-JULY 31 (80-24): THIS ROUTINE INSPECTION BY THE RESIDENT INSPECTORS OF PLANT OPERATIONS, SECURITY, RADIOLOGICAL CONTROLS, LICENSEE EVENT REPORTS (LER'S), LICENSEE ACTION ON IE BULLETINS AND CIRCULARS, REVIEW OF ILRT DATA, REACTOR VESSEL THERMAL SHOCK ISSUE, FPC ORGANIZATION CHANGES, NONROUTINE EVENTS, AND LICENSEE ACTION ON PREVIOUS INSPECTION ITEMS. NUMEROUS FACILITY TOURS WERE CONDUCTED AND SOME FACILITY OPERATIONS WERE OBSERVED ON THE BACK SHIFT. THIS REPORT ALSO INCLUDES A REVIEW OF ITEMS IDENTIFIED IN NUREG 0578 SHORT TERM LESSONS LEARNED RECOMMENDATIONS AND THE FPC SAFETY TASK FORCE. THE INSPECTION INVOLVED 500 HOURS ON SITE BY TWO RESIDENT INSPECTORS AND ONE REGION BASED INSPECTOR. TWO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED (INFRACTION - FAILURE TO OBTAIN AND ANALYZE SAMPLES FROM TEST WELL NUMBER 4 AND FAILURE TO OBTAIN COMMISSION APPROVAL PRIOR TO CHANGING ENVIRONMENTAL TECHNICAL SPECIFICATION 3.1.5; INFRACTION - FAILURE TO HAVE ADEQUATE PROCEDURES WHICH CAUSED AN OPERATIONAL EVENT RESULTING IN LOSS OF PRESSURIZER LEVEL).

INSPECTION AUGUST 11-15 (80-30): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 102 INSPECTOR-HOURS ON SITE AND AT THE CORPORATE OFFICES. OVERALL MANAGEMENT WAS REVIEWED BY INSPECTING THE QUALITY ASSURANCE PROGRAM IN THE AREAS OF LICENSEE ACTION ON PREVIOUS INSPECTION FINDINGS; QA PROGRAM - ANNUAL REVIEW; QA/QC ADMINISTRATIVE PROGRAM; ORGANIZATION AND ADMISTRATION; DESIGN, DESIGN CHANGES AND MODIFICATIONS PROGRAM; TESTS AND EXPERIMENTS PROGRAM; PROCUREMENT; AUDIT PROGRAM; HOUSEKEEPING AND CLEANLINESS PROGRAM; NONLICENSED PERSONNEL TRAINING; LICENSED OPERATOR REQUALIFICATIONS TRAINING; TRAINING RECORDS CONTROL; OFFSITE REVIEW COMMITTEE; LICENSEE ACTION ON PREVIOUSLY IDENTIFIED OPEN ITEMS; ACCEPTABILITY OF CURRENT PRACTICE FOR LOCKING CERTAIN VALVES; AND THE GASEOUS RELEASE OF AUGUST 12, 1980. OF THE 16 AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN 13 AREAS; 3 ITEMS OF NONCOMPLIANCE WERE FOUND IN 3 AREAS (INFRACTION - FAILURE TO TRANSLATE DESIGN INPUTS INTO DRAWINGS; INFRACTION - FAILURE TO FOLLOW HOUSEKEEPING PROCEDURE; DEFICIENCY - FAILURE TO DOCUMENT REVIEW OF PROCEDURES).

INSPECTION AUGUST 25-26 (80-31): THIS SPECIAL, UNANNOUNCED MANAGEMENT INSPECTION INVOLVED 13 INSPECTOR-HOURS ON SITE AND AT THE CORPORATE OFFICES IN REVIEW OF PROCEDURES CONTROLS AND ADOPTED BY FLORIDA POWER CORPORATION TO IMPLEMENT THE EVALUATION AND REPORTING REQUIREMENTS OF 10 CFR PART 21. NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

CONTINUED FROM PAGE C-12.

DAVIS-BESSE 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Ohio/Ottawa 21 miles E of Toledo
2.	Electric Reliability Council	East Central Area Reliability Coordination Agreement
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2772
5.	Design Electrical Rating (MWE Net)	906
6.	Date of Initial Criticality	August 12, 1977
7.	Date of Commercial Operation	11/21/77 (25%); 12/19/77 (40%) 01/23/78 (75%); 07/31/78 (100%)
8.	Condenser Cooling Method	Cooling Tower
9.	Condenser Cooling Water Source	Lake Erie

Utility & Contractor Information

10.	Licensee	Toledo Edison Co.
11.	Corporate Address	Edison Plaza, 300 Madison Avenue Toledo, Ohio 43652
12.	Corporate Contact	R. P. Crouse, V. P., Nuclear (419)259-5221
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	B&W
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	R. F. Warnick
19.	IE Resident Inspector	L. Reyes (419) 898-2775
20.	Licensing Project Manager	D. Garner
21.	Docket Number	50-346
22.	License Number/Date of Issue	NPF-3, 4/22/77
23.	Public Document Room	Ida Rupp Public Library 310 Madison Street Port Clinton, OH 43452

INSPECTION STATUS

ENFORCEMENT STATUS

SECTION 2.1.1 OF THE APPENDIX B ENVIRONMENTAL TECHNICAL SPECIFICATIONS REQUIRES IN PART, THAT "THE MAXIMUM DISCHARGE TEMPERATURE FOR STATION LIQUID EFFLUENTS SHALL NOT EXCEED 20 DEGREES F ABOVE AMBIENT LAKE WATER TEMPERATURE." CONTRARY TO THE ABOVE, THE 20 DEGREES F DELTA T LIMIT WAS EXCEEDED ON JANUARY 6, 7, 8, 16, AND FEBRUARY 20, 1980.
(8024 DEFICIENCY)

SECTION 3.1.2.A.4 OF THE APPENDIX B ENVIRONMENTAL TECHNICAL SPECIFICATIONS REQUIRES, IN PART, THE "ICHTHYOPLANKTON SAMPLES SHALL BE TAKEN ONCE EVERY 10 DAYS DURING THE ANTICIPATED SPAWNING SEASON, APRIL THROUGH AUGUST," AND "DUPLICATE SAMPLES SURFACE AND BOTTOM SHALL BE COLLECTED IN THE VICINITY OF...THE TOUSSAINT REEF. SAMPLES AT THE TOUSSAINT REEF SHALL BE COLLECTED ONLY DURING THE ANTICIPATED SPAWNING SEASON." CONTRARY TO THE ABOVE, NO SAMPLING WAS CONDUCTED AT THE TOUSSAINT REEF BETWEEN JUNE 1, 1980, AND JUNE 30, 1980, A PERIOD OF TIME IN EXCESS OF 10 DAYS.
(8024 DEFICIENCY)

SECTION 2.3.3 OF THE APPENDIX B ENVIRONMENTAL TECHNICAL SPECIFICATIONS REQUIRES, IN PART, THAT "THE CONCENTRATION OF SULFATE ION SHALL BE DETERMINED ONCE PER WEEK FROM A GRAB SAMPLE COLLECTED AT THE BEACH SAMPLING STATION DURING THE DISCHARGE OF THE NEUTRALIZING TANK. IF THERE IS NO DISCHARGE FROM THE NEUTRALIZING TANK, SAMPLING SHALL BE ONCE PER WEEK." SECTION 1.0 OF THE APPENDIX B ENVIRONMENTAL TECHNICAL SPECIFICATIONS DEFINES A WEEK AS "AT LEAST ONCE EVERY SEVEN DAYS WITH A MAXIMUM ALLOWABLE EXTENSION OF 25%." CONTRARY TO THE ABOVE NO SULFATE MONITORING WAS PERFORMED BETWEEN FEBRUARY 12, 1979 AND FEBRUARY 23, 1979, A PERIOD OF TIME LONGER THAN SEVEN DAYS PLUS 25%.
(8024 DEFICIENCY)

SECTION 5.4.2 OF THE APPENDIX B ENVIRONMENTAL TECHNICAL SPECIFICATIONS REQUIRES, IN PART, THAT "A REPORT SHALL BE SUBMITTED IN THE EVENT THAT A LIMITING CONDITION FOR OPERATION IS EXCEEDED (AS SPECIFIED IN SECTION 2, "LIMITING CONDITIONS FOR OPERATION")..." SECTION 2.1.1 OF THE APPENDIX B ENVIRONMENTAL TECHNICAL SPECIFICATIONS STATES "THE MAXIMUM DISCHARGE TEMPERATURE FOR STATION LIQUID EFFLUENTS SHALL NOT EXCEED 20 DEGREES F ABOVE AMBIENT LAKE WATER TEMPERATURE." CONTRARY TO THE ABOVE, ALTHOUGH THE 20 DEGREES F DELTA T LIMIT WAS EXCEEDED ON FEBRUARY 20, 1980, (DELTA T EQUALS 20.62 DEGREES F AT 0100, DELTA T EQUALS 22.09 DEGREES F AT 0200, DELTA T EQUALS 20.01 DEGREES F AT 0300, AND DELTA T EQUALS 20.54 DEGREES F AT 0800), NO REPORT WAS SUBMITTED.
(8024 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

FACILITY ITEMS

+ A strike of the Employee Office Workers Union, Local 19, continued. The strike started on July 30, 1960 and impacted the current refueling and maintenance outage schedule. Strike was settled August 18, 1980. (80-23)

MANAGERIAL ITEMS

+ On July 28, 1980 a meeting was held in RIII with management representatives of Toledo Edison Company to discuss a proposed reorganization of their corporate office. (80-23)

PLANT STATUS

The refueling/maintenance outage continued during the month.

+LAST IE SITE INSPECTION DATE: July 14 through August 15, 1980

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-52/ 01T-0	07/09/80	07/22/80	Emergency DG under-designed exhaust pipe support.
80-53/ 01T-0	07/09/80	07/22/80	Design deficiency in the SFAS sequencer.
80-59/ 03L-0	07/20/80	08/18/80	Failure of NI-1 instrument indication.
80-60/ 01T-0	08/13/80	08/26/80	Inadvertent loss of decay heat flow.
80-61/ 3L-0	08/12/80	09/10/80	Non-qualified solenoid valve operator installed on containment purge valve.

+ INSPECTION REPORT NO: 80-23

SUMMARY

INSPECTION SUMMARY

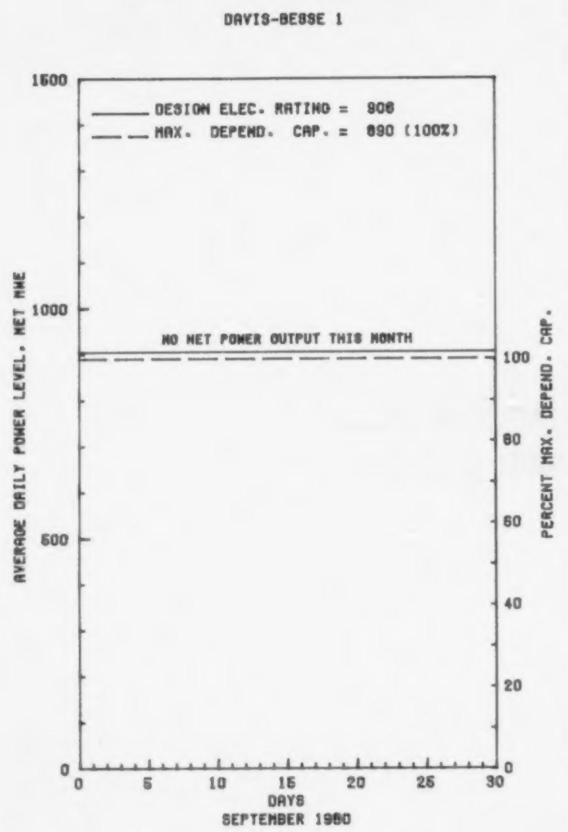
+ Inspection on July 14-18, 21-25, 28-31; August 1, 4-8 and 11-15, (80-23): Routine, unannounced inspection of followup on previous inspections' findings, monthly maintenance observation, monthly surveillance observation, Licensee Event Report followup, IE Bulletin followup, IE Circular followup, review of plant operations, inspection during long-term shutdown, confirmatory order on the Crystal River incident, implementation of TMI lessons learned, office employees union strike, loss of shutdown cooling flow on July 24, 1980, NRR technical position on location of load centers and radiation protection. The inspection involved 176 inspector-hours onsite by the Resident Inspector including 48 inspector-hours onsite during off-shifts. Of the 13 areas inspected, no items of noncompliance were identified in 12 areas; two items of apparent noncompliance were identified in the other area (failure to log the unit tours by the Operations Engineer and failure to keep records of entry and exit to vital areas).

REVIEWED BY OMPA LOEB: PE Bobe **DATE:** 11/4/80

REVIEWED BY NRR DL: **DATE:** 11/6/80

DAVIS-BESSE 1

AVERAGE POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-346
2. Utility Contact: BILAL SARSOUR (419) 259-5000 X236
3. Licensed Thermal Power (MWt): 2772
4. Nameplate Rating (Gross MWe): 1069 x 0.9 = 962
5. Design Electrical Rating (Net MWe): 906
6. Maximum Dependable Capacity (Gross MWe): 934
7. Maximum Dependable Capacity (Net MWe): 890
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	6,575.0	27,100.0
12. Hours Reactor Was Critical	.0	2,078.2	13,642.4
13. Reactor Reserve Shutdown Hours	.0	.0	2,875.8
14. Hours Generator On-Line	.0	2,008.7	11,883.4
15. Unit Reserve Shutdown Hours	.0	.0	1,728.2
16. Gross Thermal Energy (MWH)	0	4,687,304	24,886,811
17. Gross Electrical Energy (MWH)	0	1,583,559	8,307,070
18. Net Electrical Energy (MWH)	0	1,483,787	7,654,365
19. Unit Service Factor	.0	30.6	44.5
20. Unit Availability Factor	.0	30.6	51.4
21. Unit Capacity Factor (MDC Net)	.0	25.4	33.8
22. Unit Capacity Factor (DER Net)	.0	24.9	33.2
23. Unit Forced Outage Rate	.0	14.3	25.6

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 10/12/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
4	04/07/80	S	720.0	C	4				THE UNIT OUTAGE FOR SCHEDULED MAINTENANCE AND REFUELING, WHICH BEGAN ON APRIL 7, 1980, WAS STILL IN PROGRESS THROUGH THE END OF SEPTEMBER, 1980.

SUMMARY

DAVIS-BESSE REMAINED OFF-LINE FOR THE MONTH IN A CONTINUATION OF ITS REFUELING/MAINTENANCE OUTAGE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

DRESDEN 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Illinois/Grundy 9 miles E of Morris
2.	Electric Reliability Council	Mid-America Interpool Network
3.	Type of Reactor	BWR
4.	Power Level (MWT)	700
5.	Design Electrical Rating (MWE Net)	200
6.	Date of Initial Criticality	October 15, 1959
7.	Date of Commercial Operation	July 4, 1960
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Kankakee River

Utility & Contractor Information

10.	Licensee	Commonwealth Edison Co.
11.	Corporate Address	P. O. Box 767 Chicago, Illinois 60690
12.	Corporate Contact	C. Reed
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	GE
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	R. Spessard
19.	IE Resident Inspector	T. Tongue (815) 942-9267
20.	Licensing Project Manager	Paul W. O'Connor
21.	Docket Number	50-010
22.	License Number/Date of Issue	DPR-2 issued 9/28/59
23.	Public Document Room	Morris Public Library 604 Liberty Street Morris, Illinois 60451

INSPECTION STATUS

ENFORCEMENT STATUS

10 CFR 55, APPENDIX A, REQUIRES EACH LICENSED OPERATOR AND SENIOR OPERATOR TO BE COGNIZANT OF FACILITY DESIGN CHANGES, PROCEDURE CHANGES, AND FACILITY LICENSE CHANGES. THE LICENSEE'S PROCEDURE DPP-J, "LICENSED OPERATOR CONTINUING TRAINING" REQUIRES, AS PART OF THE ON-THE-JOB TRAINING, EACH LICENSED OPERATOR TO COMPLETE THE LICENSED OPERATOR REQUIRED READING, SCHEDULED BY THE TRAINING SUPERVISOR, ON A WEEKLY BASIS AND CONTAINING CHANGES IN DESIGN AND APPLICABLE PROCEDURE CHANGES. CONTRARY TO THE ABOVE, THE INSPECTOR DETERMINED THAT BETWEEN JANUARY 1980, AND JULY 1980, TEN LICENSED OPERATORS HAD NOT COMPLETED THEIR REQUIRED READING.
(8013 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

Unit 1 shutdown on October 31, 1978 for an extended outage to upgrade ECCS, chemical cleaning and refueling. Outage expected to last through Mid-1986.

LAST IE SITE INSPECTION DATE: July 7 through August 1, 1980

INSPECTION REPORT NO: 80-13

SUMMARY

INSPECTION SUMMARY

+ No inspections conducted.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE			

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 11-4-80

REVIEWED BY NRR DL: Paul W. O'Connor **DATE:** 11-6-80

DRESDEN 1

AVERAGE DAILY POWER LEVEL (MWe)

OPERATING STATUS

1. Docket Number: 50-010
2. Utility Data Prepared By. S. A. CIESZKIEWICZ (815) 942-2920 X489
3. Licensed Thermal Power (MWt): 700
4. Nameplate Rating (Gross MWe): 245.5 x 0.85 = 209
5. Design Electrical Rating (Net MWe): 200
6. Maximum Dependable Capacity (Gross MWe): 0
7. Maximum Dependable Capacity (Net MWe): 0
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons
None
9. Power Level To Which Restricted, If Any (Net MWe): None
10. Reasons For Restrictions, If Any:

SHUTDOWN INDEFINITELY

	This Month	Yr [illegible]	[illegible]ulative
11. Hours In Reporting Period			
12. Number Of Hours Reactor Was Critical			
13. Reactor Reserve Shutdown Hours			
14. Hours Generator On-[illegible]			
15. Unit Reserve Shutd[illegible]s			
16. Gross Thermal Energy Generated (MWH)			
17. Gross Electrical Energy Generated (MWH)			
18. Net Electrical Energy Generated (MWH)			
19. Unit Service Factor			
20. Unit Availability Factor			
21. Unit Capacity Factor (Using MDC Net) 1/			
22. Unit Capacity Factor (Using DER Net) 1/			
23. Unit Forced Outage Rate			

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
None

25. If Shut Down At End Of Report Period, Estimated Date of Startup: 06/86

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
SEE SUMMARY BELOW.									

SUMMARY

DRESDEN 1 IS SHUTDOWN UNTIL APPROXIMATELY JUNE, 1986 FOR THE FOLLOWING REASONS:

1) HPCI INSTALLATION FOR ECCS UPGRADING
2) EQUIPMENT UPGRADE TO IEEE 279
3) MAJOR INSERVICE INSPECTION
4) CHEMICAL CLEANING
5) REFUEL

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

NRC POWER RESTRICTIONS: NONE

DRESDEN 2

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Illinois/Grundy 9 miles E of Morris
2.	Electric Reliability Council	Mid-America Interpool Network
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2527
5.	Design Electrical Rating (MWE Net)	794
6.	Date of Initial Criticality	January 7, 1970
7.	Date of Commercial Operation	JULY, 1970
8.	Condenser Cooling Method	Cooling Lake
9.	Condenser Cooling Water Source	Kankakee and Des Plaines Rivers

tility & Contractor information

10.	Licensee	Commonwealth Edison Co.
11.	Corporate Address	P. O. Box 767 Chicago, Illinois 60690
12.	Corporate Contact	C. Reed
13.	Architect/Engineer	Sargent & Lundy
14.	Nuclear Steam Supply System	GE
15.	Constructor	United Engineers & Constructors, Inc.

ulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	R. Spessard
19.	IE Resident Inspector	T. Tongue (815) 942-9267
20.	Licensing Project Manager	Paul W. O'Connor
21.	Docket Number	50-237
22.	License Number/Date of Issue	DPR-19 issued 12/22/69
23.	Public Document Room	Morris Public Library 604 Liberty Street Morris, Illinois 60451

INSPECTION STATUS

ENFORCEMENT STATUS

10 CFR 55, APPENDIX A, REQUIRES EACH LICENSED OPERATOR AND SENIOR OPERATOR TO BE COGNIZANT OF FACILITY DESIGN CHANGES, PROCEDURE CHANGES, AND FACILITY LICENSE CHANGES. THE LICENSEE'S PROCEDURE DPP-J, "LICENSED OPERATOR CONTINUING TRAINING" REQUIRES, AS PART OF THE ON-THE-JOB TRAINING, EACH LICENSED OPERATOR TO COMPLETE THE LICENSED OPERATOR REQUIRED READING, SCHEDULED BY THE TRAINING SUPERVISOR, ON A WEEKLY BASIS AND CONTAINING CHANGES IN DESIGN AND APPLICABLE PROCEDURE CHANGES. CONTRARY TO THE ABOVE, THE INSPECTOR DETERMINED THAT BETWEEN JANUARY 1980, AND JULY 1980, TEN LICENSED OPERATORS HAD NOT COMPLETED THEIR REQUIRED READING.
(8014 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

FACILITY ITEMS

+ Commonwealth Edison has committed to develop and install an underdrain collection system at the toe of the North Dike to minimize the effects of seepage. The system is to be installed by end of 1980. (80-15)

PLANT STATUS

+ Unit operating routinely in coastdown mode.

+LAST IE SITE INSPECTION DATE: August 21, 1980

+ INSPECTION REPORT NO: 80-15

SUMMARY

INSPECTION SUMMARY

+ Health Physics Appraisal on June 18 through July 2, (80-13): Special, announced appraisal of health physics program, including organization, management, training and qualification, exposure control, radiological controls, surveillance, instrumentation, facilities and equipment, ALARA, and emergency response. The inspection involved approximately 540 inspector-hours onsite by five NRC inspectors. Significant weaknesses were identified in the health physics program in the areas of organization and management, training and qualificaitions, access control, contamination control, surveillance, instruments, facilities, and emergency response. Three apparent items of noncompliance were identified (failure to follow procedures; potential unrestricted area dose exceeding two millirems in one hour; and inadequate control of high radiation area access).

+ Meeting on August 21, (80-15): Dresden cooling lake dike seepage and remedial measures to be taken to preclude further seepage and effects on the riverbanks and dike structure.

REPORTS RECEIVED FROM LICENSEE

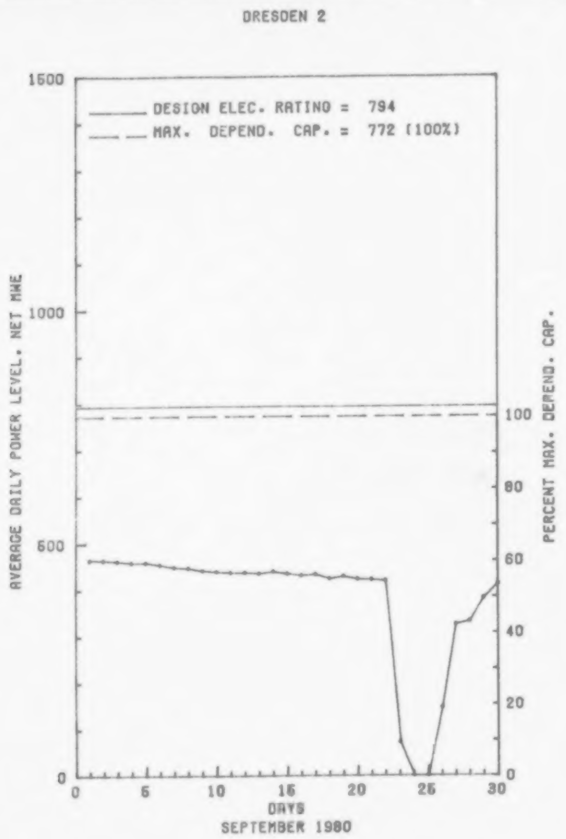
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-30/03L-0	08/18/80	09/15/80	During torus cooling operations, 2A LPCI pump mechanical seal was found leaking.

REVIEWED BY OMPA LOEB: John J. Rizzo DATE: 11-4-80

REVIEWED BY NRR DL: Paul W O'Connor DATE: 11/5/80

DRESDEN 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-237
2. Utility Contact: S. A. CIESZKIEWICZ (815) 942-2920 X489
3. Licensed Thermal Power (MWt): 2527
4. Nameplate Rating (Gross MWe): 920 x 0.9 = 828
5. Design Electrical Rating (Net MWe): 794
6. Maximum Dependable Capacity (Gross MWe): 812
7. Maximum Dependable Capacity (Net MWe): 772
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	6,575.0	91,031.0
12. Hours Reactor Was Critical	704.4	6,371.7	71,108.4
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	641.9	6,243.4	67,506.1
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	978,210	13,142,113	13?,783,945
17. Gross Electrical Energy (MWH)	291,740	4,107,690	43,114,311
18. Net Electrical Energy (MWH)	270,200	3,880,347	41,768,459
19. Unit Service Factor	89.2	95.0	74.2
20. Unit Availability Factor	89.2	95.0	74.2
21. Unit Capacity Factor (MDC Net)	48.6	76.4	58.0
22. Unit Capacity Factor (DER Net)	47.3	74.3	56.4
23. Unit Forced Outage Rate	10.8	1.5	12.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
7	10/23/80	F	78.1	A	3				REACTOR SCRAMMED ON LOW VACUUM - EXCESSIVE AIR IN-LEAKAGE INTO CONDENSER FROM "C" TURBINE HOOD; ALSO, "B" AIR EJECTOR MAIN STEAM SUPPLY LIFTED AND EXHAUSTED TO CONDENSER.

SUMMARY

DRESDEN 2 OPERATED NORMALLY FOR THE MONTH, WITH ONE OUTAGE ON THE 23RD FOR CONDENSER AIR LEAKAGE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

DRESDEN 3

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Illinois/Grundy 9 miles E of Morris
2.	Electric Reliability Council	Mid-America Interpool Network
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2527
5.	Design Electrical Rating (MWE Net)	794
6.	Date of Initial Criticality	January 31, 1971
7.	Date of Commercial Operation	November 16, 1971
8.	Condenser Cooling Method	Cooling Lake
9.	Condenser Cooling Water Source	Kankakee and Des Plaines Rivers

Utility & Contractor Information

10.	Licensee	Commonwealth Edison Co.
11.	Corporate Address	P. O. Box 767 Chicago, Illinois 60690
12.	Corporate Contact	Cordell Reed, Vice President
13.	Architect/Engineer	Sargent & Lundy
14.	Nuclear Steam Supply System	GE
15.	Constructor	United Engineers & Constructors, Inc.

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	R. Spessard
19.	IE Resident Inspector	T. Tongue 815-942-9267
20.	Licensing Project Manager	R. Bevan
21.	Docket Number	50-249
22.	License Number/Date of Issue	DPR-25 issued 1/12/71
23.	Public Document Room	Morris Public Library 604 Liberty Street Morris, Illinois 60451

INSPECTION STATUS

ENFORCEMENT STATUS

10 CFR 55, APPENDIX A, REQUIRES EACH LICENSED OPERATOR AND SENIOR OPERATOR TO BE COGNIZANT OF FACILITY DESIGN CHANGES, PROCEDURE CHANGES, AND FACILITY LICENSE CHANGES. THE LICENSEE'S PROCEDURE DPP-J, "LICENSED OPERATOR CONTINUING TRAINING" REQUIRES, AS PART OF THE ON-THE-JOB TRAINING, EACH LICENSED OPERATOR TO COMPLETE THE LICENSED OPERATOR REQUIRED READING, SCHEDULED BY THE TRAINING SUPERVISOR, ON A WEEKLY BASIS AND CONTAINING CHANGES IN DESIGN AND APPLICABLE PROCEDURE CHANGES. CONTRARY TO THE ABOVE, THE INSPECTOR DETERMINED THAT BETWEEN JANUARY 1980, AND JULY 1980, TEN LICENSED OPERATORS HAD NOT COMPLETED THEIR REQUIRED READING.
(8018 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

FACILITY ITEMS

+ Commonwealth Edison has committed to develop and install an underdrain collection system at the toe of the North Dike to minimize the effects of seepage. The system is to be installed by end of 1980. (80-19)

PLANT STATUS

Unit operating routinely.

+LAST IE SITE INSPECTION DATE: August 21, 1980

+ INSPECTION REPORT NO: 80-19

SUMMARY

INSPECTION SUMMARY

+ Health Physics Appraisal on June 18 through July 2, (80-17): Special, announced appraisal of health physics program, including organization, management, training and qualification, exposure control, radiological controls, surveillance, instrumentation, facilities and equipment, ALARA, and emergency response. The inspection involved approximately 540 inspector-hours onsite by five NRC inspectors. Significant weaknesses were identified in the health physics program in the areas of organization and management, training and qualificaitions, access control, contamination control, surveillance, instruments, facilities, and emergency response. Three apparent items of noncompliance were identified (failure to follow procedures; and inadequate control of high radiation area access).

+ Meeting on August 21, (80-19): Dresden cooling lake dike seepage and remedial measures to be taken to preclude further seepage and effects on the riverbanks and dike structure.

REPORTS RECEIVED FROM LICENSEE

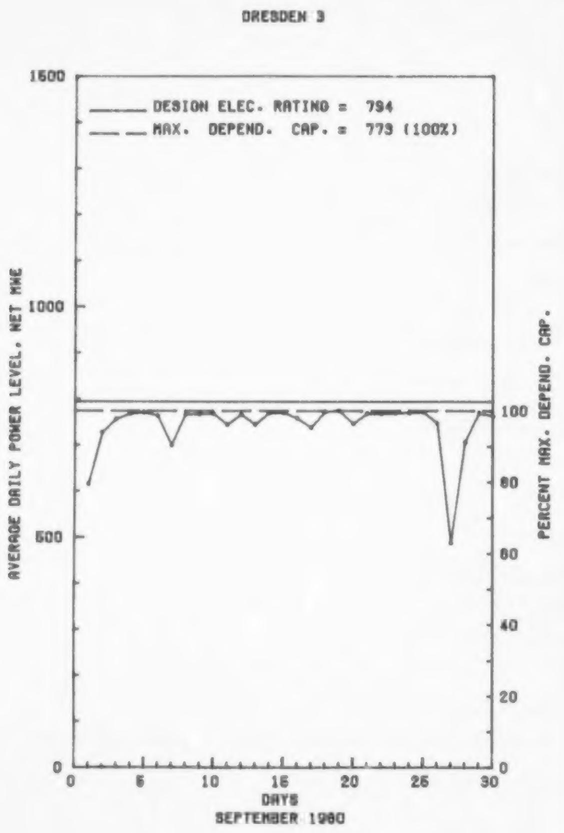
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE			

REVIEWED BY OMPA LOEB: John J. Rizzo DATE: 11-4-80

REVIEWED BY NRR DL: DATE: 11/6/80

DRESDEN 3

AVERAGE DAILY POWER LEVEL (MWe)



OPERATI NG STATUS

1. Docket Number: 50-249
2. Utility Contact: S. A. CIES EWICZ (815) 942-2920 X489
3. Licensed Thermal Power (MWt) 2527 .
4. Nameplate Rating (Gross MWe) 920 x 0.9 = 828
5. Design Electrical Rating (Ne We): 794
6. Maximum Dependable Capacity oss MWe): 812
7. Maximum Dependable Capacity t MWe): 773
8. If Changes Occur in Capacity tings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restric , If Any (het MWe): NONE
10. Reasons for Restrictions, If y:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	6,575.0	80,616.0
12. Hours Reactor Was Critical	720.0	4,346.4	60,111.8
13. Reactor Reserve Shutdown Hou	.0	.0	.0
14. Hours Generator On-Line	720.0	4,255.4	57,580.0
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,744,487	9,440,253	111,711,921
17. Gross Electrical Energy (MWH	559,047	3,012,445	36,442,119
18. Net Electrical Energy (MWH)	530,760	2,847,980	34,541,191
19. Unit Service Factor	100.0	64.7	71.4
20. Unit Availability Factor	100.0	64.7	71.4
21. Unit Capacity Factor (MDC Ne	95.4	56.0	55.4
22. Unit Capacity Factor (DER Ne	92.8	54.6	54.0
23. Unit Forced Outage Rate	.0	2.7	15.6

24. Shutdowns Scheduled Over Nex Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Repor eriod, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	NONE								

SUMMARY

DRESDEN III OPERATED AT 95% POWER FOR SEPTEMBER WITH NO OUTAGES OR REDUCTIONS REPORTED. LOAD DROPPED ON THE 27th FOR A SHORT WHILE FOR TESTING.

LEGEND

[1]Type:
F–Forced
S–Scheduled

[2]Reason:
A–Equipment Failure
B–Maintenance or Test
C–Refueling
D–Regulatory Restriction
E–Operator Training & License Examination
F–Administrative
G–Operational Error
H–Other

[3]Method:
1–Manual
2–Manual Scram
3–Automatic Scram
4–Continuations
5–Load Reduction
9–Other

[4]Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5]Exhibit H:
Same source.

DUANE ARNOLD

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Iowa/Linn 8 miles NW of Cedar Rapids
2.	Electric Reliability Council	Mid-Continent Area Reliability Coordination Agreement
3.	Type of Reactor	BWR
4.	Power Level (MWT)	1658
5.	Design Electrical Rating (MWE Net)	538
6.	Date of Initial Criticality	March 23, 1974
7.	Date of Commercial Operation	February 1, 1975
8.	Condenser Cooling Method	Cooling Towers
9.	Condenser Cooling Water Source	Cedar Rapids River Circulating Water System

Utility & Contractor Information

10.	Licensee	Iowa Electric Light and Power
11.	Corporate Address	IE Towers, P. O. Box 351 Cedar Rapids, Iowa 52406
12.	Corporate Contact	D. Arnold, President
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	GE
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	W. S. LITTLE
19.	IE Resident Inspector	W. F. CHRISTIANSON
20.	Licensing Project Manager	K. Eccleston
21.	Docket Number	50-331
22.	License Number/Date of Issue	DPR-49 issued 2/22/74
23.	Public Document Room	Reference Service Cedar Rapids Public Library 428 Third Avenue, S.E. Cedar Rapids, Iowa 52401

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

Unit operating routinely.

◆LAST IE SITE INSPECTION DATE: August 19-22 and September 4-5, 1980

◆ INSPECTION REPORT NO: 80-14

SUMMARY

INSPECTION SUMMARY

◆ Inspection on August 19-22 and September 4-5, (80-14): Licensee and contractors specifications, procedures, work activities, and QA documentation related to torus support reinforcement installation and welding. The inspection involved 46 inspector-hours onsite by one NRC inspector. No items of noncompliance or deviations were identified.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-20/ 03L-0	05/23/80	06/20/80	Emergency bus time delay relay times greater than 8.5 secs.
80-21/ 03L-0	05/24/80	06/20/80	HPCI turbine would not reach rated speed.
80-22/ 03L-0	05/28/80	06/26/80	Low oil level in "B" pump motor.
80-24/ 03L-0	06/03/80	06/26/80	RCIC inoperable.
80-25/ 03L-0	06/06/80	07/03/80	Instrument drift.
80-26/ 03L-0	06/17/80	07/03/80	Undervoltage relay in violation of TS.
80-28/ 01T-0	07/12/80	07/25/80	Scram discharge volume drain valve would not seal.
80-29/ 01T-0	07/12/80	07/25/80	RCIC system suppression chamber water level increased.
80-30/ 01T-0	07/22/80	08/05/80	River water supply system traveling screen found inoperable.
80-31/ 03L-0	07/09/80	08/07/80	LPCI Loop Selection logic would not reset.
80-32/ 03L-0	07/08/80	08/07/80	Emergency DG unresponsive to local or remote governer control.
80-33/ 03L-0	07/11/80	08/07/80	RCIC turbine trip throttle valve would not open automatically.
80-34/ 03L-0	07/12/80	08/11/80	Monthly surveillance testing of recirculation pump trip system was not completed as required by TS.
80-36/ 03L-0	07/18/80	08/15/80	RBM failed "inop".
80-37/ 03L-0	07/22/80	08/21/80	Control building air intake monitor unresponsive to source check.
80-38/ 03L-0	08/01/80	08/29/80	Particulate radiation recorders were found downscale.
80-39/ 03L-0	08/20/80	09/15/80	Core spray system header to top of core plate high differential pressure switch tripped.
80-40/ 03L-0	08/12/80	09/10/80	River water supply loop tagged out to interchange motors on pumps.
80-41/ 03L-0	08/20/80	09/10/80	Control building exhaust isolation damper would not close.
80-42/ 03L-0	08/13/80	09/10/80	Instrument drift.
80-43/ 03L-0	08/13/80	09/10/80	Turbine trip throttle valve would not close.

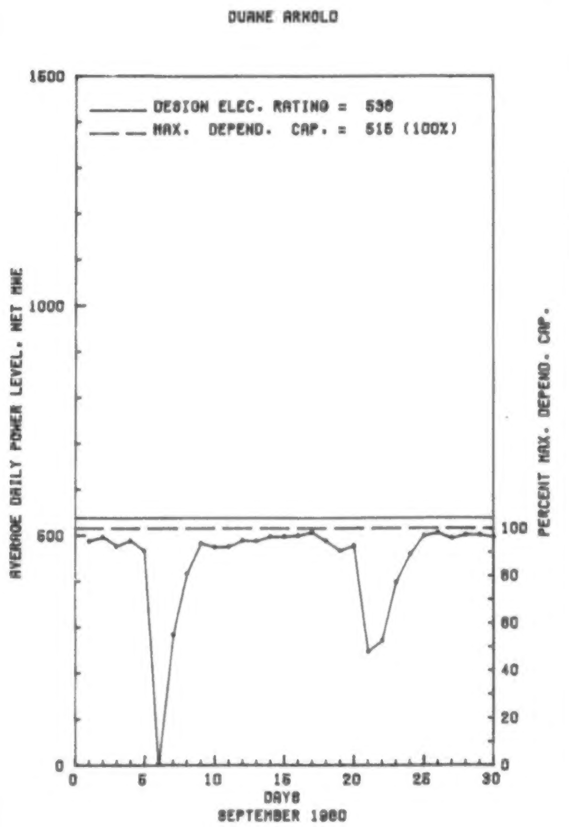
==

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 11-4-80

REVIEWED BY NRR DL: Kenneth T. Eccleston **DATE:** 11-6-80

DUANE ARNOLD

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-331
2. Utility Contact: J. Van Sickel (319) 851-5611
3. Licensed Thermal Power (MWt): 1658
4. Nameplate Rating (Gross MWe): 663 x 0.9 = 597
5. Design Electrical Rating (Net MWe): 538
6. Maximum Dependable Capacity (Gross MWe): 545
7. Maximum Dependable Capacity (Net MWe): 515
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	6,575.0	49,655.0
12. Hours Reactor Was Critical	696.5	4,619.4	35,183.5
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	692.2	4,506.7	34,325.8
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,028,448	5,980,320	43,109,880
17. Gross Electrical Energy (MWH)	340,243	1,994,052	14,418,418
18. Net Electrical Energy (MWH)	320,012	1,868,552	13,483,616
19. Unit Service Factor	96.1	68.5	69.1
20. Unit Availability Factor	96.1	68.5	69.1
21. Unit Capacity Factor (MDC Net)	86.3	55.2	52.7
22. Unit Capacity Factor (DER Net)	82.6	52.8	50.5
23. Unit Forced Outage Rate	3.9	7.3	19.3

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
REFUELING - MARCH 7, 1981, - 9 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	License Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
17	09/06/80	F	19.0	A	1		HA	TURBIN	POWER WAS REDUCED AND THE TURBINE TAKEN OFF-LINE TO REPAIR AN OVERSPEED TRIP OIL LINE.
18	09/21/80	F	8.8	G	3		CD	VALVEX	REACTOR SCRAMMED FROM MSIV'S BEING LESS THAN 90% OPEN DUE TO LOSS OF NITROGEN PRESSURE. A GROUP ISOLATION EARLIER IN THE DAY HAD NOT BEEN PROPERLY RESET.

SUMMARY

DUANE ARNOLD OPERATED AT APPROXIMATELY 95% POWER IN SEPTEMBER, WITH AN OUTAGE ON THE 6TH FOR EQUIPMENT FAILURE AND ONE ON THE 21ST FOR OPERATIONAL ERROR.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

FARLEY 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1. Location (state/county) (nearest population center)	Alabama/Houston 24 miles SE of Dothan
2. Electric Reliability Council	Southeastern Electric Reliability Council
3. Type of Reactor	PWR
4. Power Level (MWT)	2652
5. Design Electrical Rating (MWE Net)	829
6. Date of Initial Criticality	August 9, 1977
7. Date of Commercial Operation	December 1, 1977
8. Condenser Cooling Method	Once Through With Forced Draft Cooling Towers
9. Condenser Cooling Water Source	Chattahoochee River

Utility & Contractor Information

10. Licensee	Alabama Power Company
11. Corporate Address	600 North 18th Street Birmingham, Alabama
12. Corporate Contact	F. L. Clayton, Jr. Senior Vice President (205-323-5341)
13. Architect/Engineer	Bechtel/SSI
14. Nuclear Steam Supply System	Westinghouse
15. Constructor	Daniel

Regulatory Information

16. IE Region responsible	II
17. IE Branch Chief	R. C. Lewis
18. IE Section Chief	R. D. Martin
19. IE Resident Inspector	W. H. Bradford (205-899-3386)
20. Licensing Project Manager	E. A. Reeves
21. Docket Number	50-348
22. License Number/Date of Issue	NPF-2, 6/25/77
23. Public Document Room	G. S. Houston Memorial Library 212 W. Berdeshaw Street Dothan, Alabama 36301

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO SECTIONS 6.2.2.F AND 6.4.2 OF THE TECHNICAL SPECIFICATIONS, PERSONNEL FROM THE OPERATIONS GROUP WHO ARE NOT INCLUDED ON THE TRAINING SUPERVISOR'S "QUALIFIED" FIRE BRIGADE ROSTER ARE ROUTINELY ASSIGNED TO PERFORM THE PLANT FIRE BRIGADE FUNCTIONS.
(8020 INFRACTION)

MEMBERS OF THE SECURITY FORCE WERE NOT PROPERLY QUALIFIED TO PERFORM SECURITY FUNCTIONS.
(8021 INFRACTION)

AS REQUIRED BY SECTION 6.8.1 OF THE TECHNICAL SPECIFICATION, WRITTEN PROCEDURES SHALL BE ESTABLISHED, IMPLEMENTED AND MAINTAINED. CONTRARY TO THE ABOVE ON APRIL 23, 1980 AND MAY 23, 1980 ERRORS WERE MADE IN FNP-1-STP-21.1 "MAIN STEAM LINE ISOLATION VALVE OPERABILITY TEST (PARTIAL CLOSURE)". THE ERRORS HAD BEEN CROSSED OUT AND NEW DATA INSERTED. THE CORRECTED DATA HAD NOT BEEN INITIALED AND DATED AS REQUIRED BY SECTION 6.0 OF ADMINISTRATIVE PROCEDURE NO. 16, "CONDUCT OF OPERATIONS - OPERATIONS GROUP".
(8022 DEFICIENCY)

AS REQUIRED BY TECHNICAL SPECIFICATIONS 6.8.1 AND 6.8.2, WRITTEN PROCEDURES SHALL BE ESTABLISHED, IMPLEMENTED AND MAINTAINED COVERING THE APPLICABLE PROCEDURES RECOMMENDED IN APPENDIX "A" OF REGULATORY GUIDE 1.33, NOVEMBER, 1972 AND EACH PROCEDURE AND ADMINISTRATIVE PROCEDURE INCLUDING TEMPORARY CHANGES SHALL BE REVIEWED PRIOR TO IMPLEMENTATION AS SET FORTH IN TECHNICAL SPECIFICATION 6.5. APPENDIX A OF REGULATORY GUIDE 1.33 STATES IN PART THAT CHEMICAL AND RADIOCHEMICAL PROCEDURES SHOULD BE WRITTEN TO PRESCRIBE THE NATURE AND FREQUENCY OF SAMPLING AND ANALYSES AND THAT THESE PROCEDURES SHOULD SPECIFY LABORATORY INSTRUCTIONS AND CALIBRATION OF LABORATORY EQUIPMENT. CONTRARY TO THE ABOVE, DURING THE PERIOD OF AUGUST 1978 TO AUGUST, 1980, UNREVIEWED CHANGES WERE MADE IN THE CALIBRATION STEP OF PROCEDURE FNP-0-CP-04, POTENTIOMETRIC DETERMINATION OF FLUORIDE ION IN AQUEOUS SOLUTIONS.
(8024 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

REFUELING OUTAGE SCHEDULED TO BEGIN NOVEMBER 7, 1980.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

NUMEROUS OUTAGES AND REDUCTIONS DURING MONTH.

+ LAST IE SITE INSPECTION DATE: SEPTEMBER 1-30, 1980

+ INSPECTION REPORT NO: 50-348/80-29

SUMMARY

INSPECTION SUMMARY

+ INSPECTION AUGUST 4-7 (80-20): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 34 INSPECTOR-HOURS ON SITE IN THE AREAS OF FIRE PROTECTION/PREVENTION. OF THE AREAS INSPECTED, ONE ITEM OF NONCOMPLIANCE WAS FOUND (INFRACTION - ASSIGNMENT OF UNQUALIFIED PERSONNEL BY OPERATIONS GROUP TO PERFORM FIRE BRIGADE FUNCTIONS; AND NO APPARENT DEVIATIONS WERE FOUND.

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-044/03L-0	07/23/80	08/22/80	DIESEL GENERATOR 1C FAILED TO REACH RATED SPEED AND VOLTAGE.
80-045/03L-0	07/27/80	08/26/80	THE CONTAINMENT ATMOSPHERE RADIOACTIVITY AND GASEOUS RADIOACTIVITY MONITORING SYSTEMS WERE INOPERABLE.
80-046/03L-0	08/13/80	09/08/80	STEAM FLOW CHANNEL 475 FAILED DUE TO A DEFECTIVE POWER SUPPLY CARD.
80-047/03L-0	08/12/80	09/11/80	THE SERVICE WATER BUILDING B TRAIN BATTERY SYSTEM WAS INOPERABLE.

INSPECTION JULY 1-31 (80-22): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 350 INSPECTOR-HOURS ON SITE IN THE AREAS OF UNIT NO. 1 PLANT OPERATION, LICENSEE EVENT REPORTS, OPERATIONAL STAFFING, PLANT SAFETY COMMITTEE, PLANT PROCEDURES, UNIT NO. 2 PREOPERATIONAL TEST PROGRAM, WITNESS OF PREOPERATIONAL TEST, UNIT NO. 2 PLANT TOUR, LICENSEE'S STRIKE CONTINGENCY PLAN. OF THE NINE AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN EIGHT AREAS; ONE APPARENT ITEM OF NONCOMPLIANCE WAS FOUND IN ONE AREA (FAILURE TO FOLLOW PROCEDURES).

INSPECTION AUGUST 14-15 (80-23): THIS SPECIAL, UNANNOUNCED INSPECTION INVOLVED NINE INSPECTOR-HOURS ON SITE IN THE AREAS OF FOLLOWUP OF ITEMS OF NONCOMPLIANCE AND INSPECTOR FOLLOWUP ITEMS RELATIVE TO PART 21 EVALUATION AND REPORTING MATTERS. NONCOMPLIANCE, "PART 21 EVALUATION" AND INSPECTOR FOLLOWUP ITEM, "PART 21 PROCEDURES REVISION" ARE CLOSED.

INSPECTION AUGUST 12-14 (80-31): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 18 INSPECTOR-HOURS ON SITE IN THE AREAS OF QUALITY CONTROL AND CONFIRMATORY MEASUREMENTS INCLUDING: REVIEW OF THE LABORATORY QUALITY CONTROL PROGRAM; REVIEW OF CHEMISTRY AND RADIOCHEMISTRY PROCEDURES; AND COMPARISON OF THE RESULTS OF SPLIT SAMPLES ANALYZED BY THE LICENSEE AND THE NRC REGION II MOBILE LABORATORY. OF THE THREE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN TWO AREAS; ONE ITEM OF NONCOMPLIANCE WAS FOUND IN ONE AREA (DEFICIENCY - FAILURE TO REVIEW CHANGES IN CHEMISTRY PROCEDURE FNP-0-CPP-4, DETERMINATION OF FLUORIDE ION IN AQUEOUS SOLUTIONS).

INSPECTION AUGUST 27-29 (80-25): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 32 INSPECTOR-HOURS ON SITE IN THE AREAS OF MANAGEMENT CONTROL OF THE RADIOLOGICAL ENVIRONMENTAL MONITORING PROGRAM, QUALITY CONTROL OF ANALYTICAL MEASUREMENTS, INSTALLATION AND OPERABILITY OF RADIOLOGICAL ENVIRONMENTAL MONITORING STATIONS, REVIEW OF RADIOLOGICAL ENVIRONMENTAL MONITORING REPORTS AND DATA, REVIEW OF RADIOLOGICAL ENVIRONMENTAL MONITORING PROGRAM. OF THE SIX AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

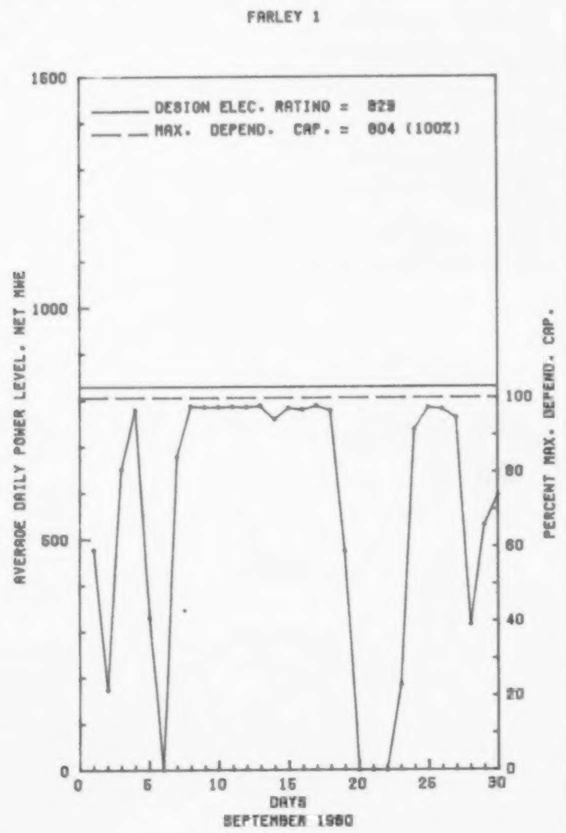
INSPECTION AUGUST 1-29 (80-27): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 284 INSPECTOR-HOURS ON SITE IN THE AREAS OF UNIT 1 PLANT OPERATION, PLANT SECURITY, UNIT 2 PLANT TOUR, OPEN ITEMS PREOPERATIONAL TEST WITNESSING, UNIT NO. 2 PLANT OPERATIONAL STATUS, PREOPERATIONAL TEST STATUS, NEAR TERM OPERATING LICENSEE REQUIREMENTS, AND PLANT MAINTENANCE. OF THE NINE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

REVIEWED BY OMPA LOEB: Bobe **DATE:** 11/4/80

REVIEWED BY NRR DL: Reeves **DATE:** 11/6/80

FARLEY 1

AVERAGE POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-348
2. Utility Contact: W. G. Hairston,III (205) 899-5156
3. Licensed Thermal Power (MWt): 2652
4. Nameplate Rating (Gross MWe): 1045 x 0.85 = 888
5. Design Electrical Rating (Net MWe): 829
6. Maximum Dependable Capacity (Gross MWe): 845
7. Maximum Dependable Capacity (Net MWe): 804
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	6,575.0	24,839.0
12. Hours Reactor Was Critical	625.2	5,332.8	16,268.3
13. Reactor Reserve Shutdown Hours	94.8	1,041.5	2,446.8
14. Hours Generator On-Line	593.6	5,199.9	15,790.4
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,398,265	13,075,375	39,673,390
17. Gross Electrical Energy (MWH)	437,680	4,153,640	12,677,678
18. Net Electrical Energy (MWH)	402,214	3,909,446	11,913,454
19. Unit Service Factor	82.4	79.1	63.6
20. Unit Availability Factor	82.4	79.1	63.6
21. Unit Capacity Factor (MDC Net)	69.5	74.0	59.7
22. Unit Capacity Factor (DER Net)	67.4	71.7	57.9
23. Unit Forced Outage Rate	17.6	7.1	7.7

24. Shutdowns Scheduled Over Next 6 Months (Type,Date,and Duration of Each):
REFUELING OUTAGE - EARLY NOVEMBER 1980 - APPROX. 8 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
060	09/01/80	F	.0	B	3		HH	VALVEX	UNIT POWER REDUCED AND HELD AT 125 MW TO REPAIR AN AIR LEAK ON "A" MAIN FEED REGULATING VALVE DIAPHRAM.
061	09/01/80	F	3.8	G	3		HH	VALVEX	UNIT TRIPPED ON "1C" S/G HIGH LEVEL. WHILE INCREASING POWER, "B" SGFP WAS LEFT IN MANUAL AND "1C" S/G LEVEL WAS DECREASING. WHEN "1C" S/G FEED WAS INCREASED, A HIGH LEVEL OCCURRED. OPERATOR ERROR.
062	09/02/80	F	6.8	G	3		HH	VALVEX	UNIT TRIPPED ON "1B" S/G LO-LO LEVEL AFTER TRANSFER TO FEED REGULATING VALVE BYPASS. OPERATOR ERROR.
063	09/05/80	F	22.3	H	3		EB	CKTBRK	UNIT TRIPPED WHEN INCOMING BREAKER FROM 4160 BUS 1F TO LOAD CENTER 1D OPENED. THIS GAVE THE SSPS A RCP "BREAKER OPEN" SIGNAL. CAUSE UNKNOWN.
064	09/06/80	F	11.5	G	3		HH	VALVEX	UNIT TRIPPED ON "1C" S/G LO-LO LEVEL AFTER TRANSFER TO FEED REGULATING VALVE BYPASS. OPERATOR ERROR.
065	09/14/80	S	.0	B	5		HA	VALVEX	UNIT POWER REDUCED TO AND HELD AT 723 MW FOR GOVERNOR VALVE TESTING.
066	09/18/80	F	.0	B	5		HJ	PUMPXX	UNIT POWER REDUCED TO AND HELD AT 770 MW TO TAKE "1B" HEATER DRAIN PUMP OFF-LINE. BEARINGS WERE OVERHEATING.
067	09/19/80	F	82.0	F	1		SD	PENETR	UNIT MANUALLY TAKEN OFF-LINE TO INVESTIGATE POSSIBLE CONTAINMENT EQUIPMENT HATCH BOLT DAMAGE DUE TO OVERTORQUING. WRONG TORQUE VALUE IN VENDOR MANUAL.
068	09/23/80	F	.0	B	5		CB	PUMPXX	UNIT POWER REDUCED TO AND HELD AT 117 MW WHILE INCREASING POWER AFTER OUTAGE TO INVESTIGATE "1A" RCP BEARING OIL RESERVOIR HIGH LEVEL.
069	09/23/80	F	.0	B	5		HC	ZZZZZZ	UNIT POWER REDUCED TO AND HELD AT 50 MW WHILE INCREASING POWER AFTER OUTAGE TO INVESTIGATE LOW CONDENSER VACUUM.
070	09/27/80	S	.0	B	5		HA	VALVEX	UNIT POWER REDUCED TO AND HELD AT 730 MW FOR GOVERNOR VALVE TESTING.
071	09/28/80	F	.0	B	5		HC	ZZZZZZ	UNIT POWER REDUCED TO AND HELD AT VARIOUS LEVELS (70 MW TO 650 MW) TO INVESTIGATE LOW CONDENSER VACUUM.

SUMMARY

FARLEY OPERATED NORMALLY DURING SEPTEMBER WITH NUMEROUS OUTAGES AND REDUCTIONS LISTED IN DETAIL ABOVE

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

NRC POWER RESTRICTIONS: NONE

FITZPATRICK

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	New York/Oswego 8 miles NE of Oswego
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2436
5.	Design Electrical Rating (MWE Net)	821
6.	Date of Initial Criticality	November 17, 1974
7.	Date of Commercial Operation	July 28, 1975
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Lake Ontario

Utility & Contractor Information

10.	Licensee	Power Authority of the State of New York
11.	Corporate Address	10 Columbus Circle New York, New York 10019
12.	Corporate Contact	J. P. Bayne, Manager of Nuclear Operations
13.	Architect/Engineer	Stone & Webster
14.	Nuclear Steam Supply System	GE
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Section Chief	H. B. KISTER
19.	IE Resident Inspector	W. H. BAUNACK (315) 342-4907 FTS: 264-3311
20.	Licensing Project Manager	P. Polk (X7872)
21.	Docket Number	50-333
22.	License Number/Date of Issue	DPR-59, 10/17/74
23.	Public Document Room	State University College of Oswego Penfield Library - Documents Oswego, NY 13126 (315)341-2323

INSPECTION STATUS

ENFORCEMENT STATUS

FAILURE TO MARK PACKAGE RADIOACTIVE LSA.
(8014 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

+ MANAGERIAL ITEMS

+ THE LICENSEE TERMINATED ITS CONTRACT FOR THE SECURITY FORCE WITH PINKERTON AND IS IN THE PROCESS OF FORMING ITS OWN SECURITY FORCE.

PLANT STATUS:

+ ROUTINE POWER OPERATION WITH THE RECOMBINER OUT OF SERVICE. REPRESENTATIVE OFFGAS AND STACK RELEASE RATES ARE 6000 AND 2268 MICROCURIES PER SECOND RESPECTIVELY.

+ LAST IE SITE INSPECTION DATE: 9/22-23/80

+ INSPECTION REPORT NO: 50-333/80-18

SUMMARY

INSPECTION SUMMARY

+ 50-333/80-13 - AUG 18-22: ROUTINE, UNANNOUNCED INSPECTION BY ONE REGION-BASED INSPECTOR (28 HRS) OF: SECURITY PLAN, SECURITY PROGRAM AUDIT; SECURITY ORGANIZATION; PHYSICAL BARRIERS (PROTECTED AREA) (VITAL AREA); LIGHTING; ACCESS CONTROL (IDENTIFICATION, AUTHORIZATION AND BADGING) (SEARCH); (ESCORTING); (VITAL AREAS); ALARM STATIONS; DETECTION AIDS; ASSESSMENT AIDS; COMMUNICATIONS; TESTING AND MAINTENANCE; COMPENSATORY MEASURES; POWER SUPPLY; RESPONSE; KEYS, LOCKS AND COMBINATIONS; RECORDS AND REPORTS AND REVIEW OF ACTIONS ON PREVIOUS ITEMS OF NONCOMPLIANCE. NO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED.

+ 50-333/80-14 (15000039/80-14) - JUN 19: VISIT TO CHEM-NUCLEAR WASTE DISPOSAL SITE, BARNWELL, SOUTH CAROLINA BY ONE REGION-BASED INSPECTOR FOR ANNOUNCED INSPECTIONS OF LICENSEE-SHIPPERS. EACH LICENSEE-SHIPPER VEHICLE WAS INSPECTED FOR COMPLIANCE WITH DEPARTMENT OF TRANSPORTATION (DOT) AND NUCLEAR REGULATORY COMMISSION (NRC) REGULATIONS AS FOLLOWS: (1) SHIPPING PAPER REQUIREMENTS; (2) DOT PLACARDING REQUIREMENTS; (3) RADIATION LEVELS; (4) REMOVABLE CONTAMINATION; (5) DOT MARKING AND LABELING REQUIREMENTS FOR PACKAGES; (6) DOT AND NRC REQUIREMENTS FOR EXTERNAL PACKAGE FEATURES, AND (7) PROHIBITED ARTICLES OR CONTENTS. ONE ITEM OF NONCOMPLIANCE WAS IDENTIFIED: FAILURE TO MARK PACKAGE TO IDENTIFY CONTENTS.

REPORTS RECEIVED FROM LICENSEE

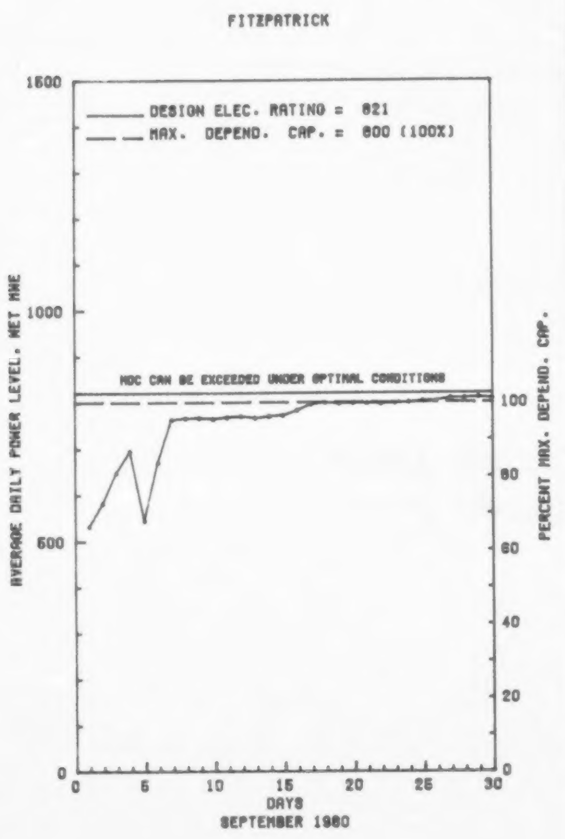
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-061/03L	07/14/80	08/12/80	HYDRAULIC SNUBBERS FAILED SURVEILLANCE TEST AS A RESULT OF NORMAL WEAR AND TEAR
80-062/03L	07/17/80	08/14/80	RHR SERVICE WATER PUMP FAILED SURVEILLANCE TEST BECAUSE SEASONAL ORIFICE NOT REMOVED
80-063/03L	07/17/80	08/14/80	APRM FAILED SURVEILLANCE TEST BECAUSE OF INSTRUMENT DRIFT
80-064/04T	08/04/80	08/18/80	ENVIRONMENTAL RADIOLOGICAL MONITORING RESULTED IN PERIPHYTON ACTIVITY GREATER THAN 10 TIMES CONTROL SAMPLE
80-065/03T	08/14/80	08/27/80	STANDBY GAS TREATMENT TRAINS INOPERABLE BECAUSE OF WATER IN DISCHARGE LINE
80-066/03L	08/03/80	08/28/80	CORE SPRAY SYSTEM PIPE BREAK DETECTION SWITCHES FAILED SURVEILLANCE BECAUSE OF INSTRUMENT DRIFT
80-067/03L	08/05/80	08/29/80	HPCI AND RCIC STEAM LINE LOW PRESSURE SWITCHES SET ABOVE ALLOWABLE SETPOINT BECAUSE OF INSTRUMENT DRIFT
80-068/03L	08/06/80	08/29/80	DRYWELL EQUIPMENT DRAIN SUMP PUMP FLOW TRANSMITTER OUT OF TOLERANCE BECAUSE OF INSTRUMENT DRIFT
80-069/03L	08/07/80	08/29/80	ADS TIMER SETPOINT OUT OF TOLERANCE DUE TO INSTRUMENT DRIFT

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 11-4-80

REVIEWED BY NRR DL: **DATE:** 11 6 '80

FITZPATRICK

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-333
2. Utility Contact: E. Zufelt (315) 342-3840
3. Licensed Thermal Power (MWt): 2436
4. Nameplate Rating (Gross MWe): 981 x 0.9 = 883
5. Design Electrical Rating (Net MWe): 821
6. Maximum Dependable Capacity (Gross MWe): 830
7. Maximum Dependable Capacity (Net MWe): 800
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

ITEM 21 CALCULATED WITH A WEIGHTED MDC

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	6,575.0	65,408.0
12. Hours Reactor Was Critical	720.0	4,155.1	30,785.3
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	720.0	4,048.0	29,786.0
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,665,872	8,048,096	59,223,290
17. Gross Electrical Energy (MWH)	559,030	2,748,520	20,102,220
18. Net Electrical Energy (MWH)	542,145	2,660,295	19,450,870
19. Unit Service Factor	100.0	61.6	65.6
20. Unit Availability Factor	100.0	61.6	65.6
21. Unit Capacity Factor (MDC Net)	94.1	50.6	58.3
22. Unit Capacity Factor (DER Net)	91.7	49.3	52.2
23. Unit Forced Outage Rate	.0	3.9	19.0

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
SCHED SNUBBER INSPECT & MAINT OUTAGE -01/81- APPROX 10 DAYS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
16	09/05/80	S	.0	H	5				POWER REDUCED FOR ROD PATTERN CHANGE.

SUMMARY

UNIT OPERATING AT NEAR FULL POWER FOR THIS REPORTING PERIOD WITH ONE REDUCTION FOR ROD PATTERN CHANGE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

FORT CALHOUN

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Nebraska/Washington 19 miles N of Omaha
2.	Electric Reliability Council	Mid-Continent Area Reliability Coordination Agreement
3.	Type of Reactor	PWR
4.	Power Level (MWT)	1500
5.	Design Electrical Rating (MWE Net)	457
6.	Date of Initial Criticality	August 6, 1973
7.	Date of Commercial Operation	June 20, 1974
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Missouri River

Utility & Contractor Information

10.	Licensee	W. C. Jones Division Manager – Production Operations Omaha Public Power District
11.	Corporate Address	1623 Harney Street Omaha, Nebraska 68102
12.	Corporate Contact	T. E. Short Assistant General Manager
13.	Architect/Engineer	Gibbs & Hill, Inc.
14.	Nuclear Steam Supply System	Combustion Engineering
15.	Constructor	Gibbs & Hill, Durham & Richardson, Inc.

Regulatory Information

16.	IE Region responsible	IV
17.	IE Branch Chief	G. L. Madsen
18.	IE Section Chief	D. M. Hunnicutt
19.	IE Resident Inspector	D. Kelley
20.	Licensing Project Manager	P. C. Wagner
21.	Docket Number	50-285
22.	License Number/Date of Issue	DPR-40, 5/24/73
23.	Public Document Room	W. Dale Clark Library 215 S. 15th Street Omaha, Nebraska 68102

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

FACILITY ITEMS (PLANS AND PROCEDURES):

MANAGERIAL ITEMS:

PLANT STATUS:

No significant information

LAST IE SITE INSPECTION DATE: July 30 - August 31, 1980

INSPECTION REPORT NO: 50-285/80-15

SUMMARY

INSPECTION SUMMARY

Inspection on August 11-15, 1980 (80-14): Included review of:
Security Plan; Security Program Audit; Security Organization; Physi
Barriers (Protected Area); Physical Barriers (Vital Areas); Access
Control (Identification, Authorization and Badging); Access Control
(Search); Access Control (Escorting); Access Control (Vital Areas);
Alarm Stations; Communications; Testing and Maintenance; and Respon
The licensee was found to be in compliance with NRC requirements win
the 13 areas examined during the inspection.

Inspection on July 29-30, 1980 (80-13): Routine, announced inspect
of the licensee's Emergency Planning Program pertaining to emergenc
tests and drills and follow-up on a previously identified item of n
compliance. The inspector observed an emergency exercise on July 2
1980.

Inspection conducted during period of July 30 - August 31, 1980 (80):
Routine, announced inspection including Operation Safety Verificati
and start up testing - refueling. The inspection of an on-site
activity by the Resident Inspector included thirty-three (33) insper-
hours. Within the two (2) areas inspected, one item of noncomplianc
was identified (infraction - Physical Security).

REPORTS RECEIVED FROM LICENSEE

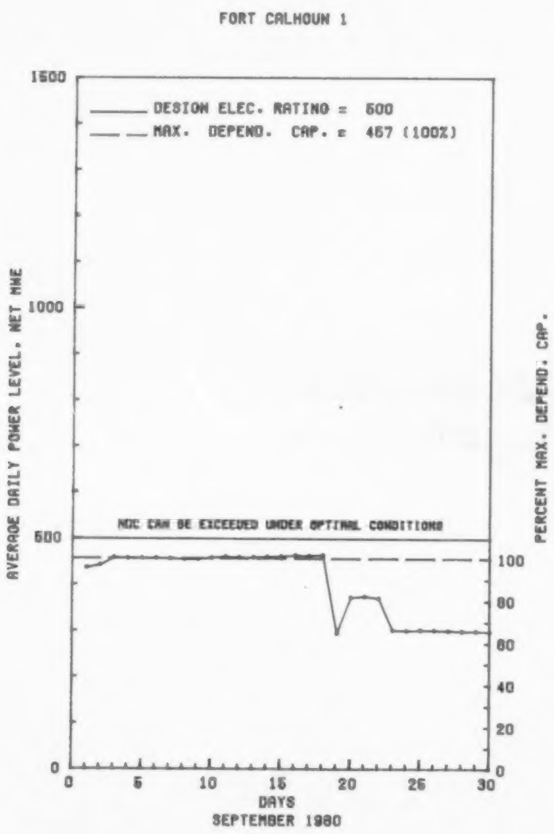
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-03 03L-0	1/22/80	2/15/80	Diesel generator zener diode failure
80-04 01T-0	1/22/80	2/6/80	Degraded fire barrier
80-05 03L-0	1/31/80	2/25/80	Rupture disc failure during ILRT
80-06 01T-0	4/15/80	4/28/80	Splice documentation not available (Ref. IEB 79-01)
80-07 01T-0	4/30/80	5/7/80	Splice documentation not available (Ref. IEB 79-01)
80-08 03L-0	5/15/80	5/19/80	RPS AB-1 matrix relay failure
80-09 99T-0	5/22/80	5/29/80	Stack iodine monitor sensitivity calibration error
80-10 01T-0	5/16/80	6/17/80	RCP closure stud corrosion
80-17 03L-0	8/4/80	8/25/80	Stack particulate setting wrong
80-18 03L-0	8/8/80	9/5/80	Small release to auxiliary building from VCT
80-19 03L-0	9/4/80	10/3/80	Seismic restraint inoperable

REVIEWED BY OMPA LOEB: **DATE:** 11/04/80

REVIEWED BY NRR DL: PC Wagner **DATE:** 11/5/80

FORT CALHOUN

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-285
2. Utility Contact: T. L. PATTERSON (402) 536-4413
3. Licensed Thermal Power (MWt): 1500
4. Nameplate Rating (Gross MWe): 591 x 0.85 = 502
5. Design Electrical Rating (Net MWe): 457
6. Maximum Dependable Capacity (Gross MWe): 481
7. Maximum Dependable Capacity (Net MWe): 457
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

ITEM 21 CALCULATED WITH A WEIGHTED MDC.

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	6,575.0	61,512.0
12. Hours Reactor Was Critical	718.6	3,147.7	47,569.3
13. Reactor Reserve Shutdown Hours	.0	.0	1,309.5
14. Hours Generator On-Line	716.4	3,103.8	46,554.9
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	935,548	4,102,294	57,337,321
17. Gross Electrical Energy (MWH)	305,248	1,333,180	18,973,410
18. Net Electrical Energy (MWH)	289,263	1,263,461	17,950,979
19. Unit Service Factor	99.5	47.2	75.7
20. Unit Availability Factor	99.5	47.2	75.7
21. Unit Capacity Factor (MDC Net)	87.9	42.0	64.3
22. Unit Capacity Factor (DER Net)	87.9	42.0	53.9
23. Unit Forced Outage Rate	.5	.4	4.2

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-03	09/19/80	F	3.6	A	3		RB	CKTBRK	CONTROL ROD CLUTCH POWER SUPPLY BREAKER TRIPPED WHEN SWITCHING POWER SOURCES TO INVESTIGATE VOLTAGE SPIKING ON INVERTER C.

SUMMARY

FORT CALHOUN 1 OPERATED NORMALLY DURING SEPTEMBER, WITH AN OUTAGE ON THE 19TH DUE TO ELECTRICAL EQUIPMENT FAILURE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B- Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

FORT ST. VRAIN

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Colorado/Weld 35 miles N of Denver (near Platteville)
2.	Electric Reliability Council	Western Systems Coordinating Council
3.	Type of Reactor	HTGR
4.	Power Level (MWT)	842
5.	Design Electrical Rating (MWE Net)	330
6.	Date of Initial Criticality	January 31, 1974
7.	Date of Commercial Operation	July 1, 1979
8.	Condenser Cooling Method	Towers
9.	Condenser Cooling Water Source	South Platte River/Shallow Wells

Utility & Contractor Information

10.	Licensee	Public Service Company of Colorado
11.	Corporate Address	P. O. Box 840 Denver, Colorado 80201
12.	Corporate Contact	C. K. Millen Senior Vice President
13.	Architect/Engineer	Sargent & Lundy
14.	Nuclear Steam Supply System	General Atomic Company
15.	Constructor	General Atomic Company

Regulatory Information

16.	IE Region responsible	IV
17.	IE Branch Chief	G. L. Madsen
18.	IE Section Chief	T. F. Westerman
19.	IE Resident Inspector	M. W. Dickerson (303) 785-2281
20.	Licensing Project Manager	G. Kuzmycz
21.	Docket Number	50-267
22.	License Number/Date of Issue	DPR-34, 12/21/73
23.	Public Document Room	Greeley Public Library City Complex Building Greeley, Colorado 80631

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO TS 7.4A, ADM 28 AND ADMIN PROCEDURE P-1, RECORDER ZR-1233-1 WAS FOUND NOT WORKING AND NO PTR HAD BEEN WRITTEN.
(8018 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

Areas related to potential equipment modification, which are under review and discussion with the NRC include: helium circulator auxiliaries; accident analysis relating to outlet temperature dispersion and orifice settings; hangers for 2" and smaller piping; and plant oscillations.

FACILITY ITEMS (PLANS AND PROCEDURES):

MANAGERIAL ITEMS:

PLANT STATUS:

Continuing plant shutdown for completion of surveillance and maintenance activities.

LAST IE SITE INSPECTION DATE: August 31, 1980

INSPECTION REPORT NO: 50-267/80-18

SUMMARY

INSPECTION SUMMARY

Inspection on August 11-15, 1980 (80-16): Routine, unannounced inspection of design, design changes and modifications, and of the non-routine reporting program. Within the two (2) areas inspected, no apparent item of noncompliance was identified in the other.

Inspection conducted on August 1-31, 1980 (80-18): Routine, announced inspection of plant operations; review of event reports; followup on previous items of noncompliance; surveillance; maintenance; operational safety verification; and review IE Bulletins/Circulars. Within the seven (7) areas inspected, one item of noncompliance was identified (Infraction - failure to comply with procedural requirements).

NRC POWER RESTRICTIONS: Restricted to 70% of Licensed Thermal Power.

REPORTS RECEIVED FROM LICENSEE

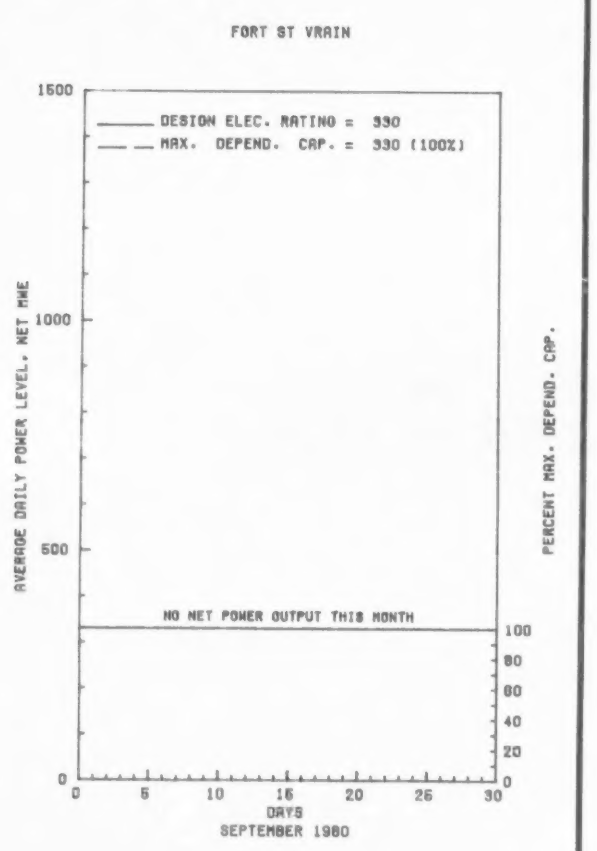
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-44 03L-0	8/16/80	9/15/80	"B" and "A" boiler feed pumps out of service (LCO 4.3.2)
80-45 03L-0	8/16/80	9/15/80	Hydraulic oil line failure bolation of loop (LCO 4.3.7)
80-46 03L-0	8/20/80	9/19/80	Operation at power with water temperature of PCRV greater than 120 deg f. (LCO 4.2.15b)
80-47 03L-0	8/25/80	9/24/80	16 snubbers found to be inoperable (LCO 4.3.10)

REVIEWED BY OMPA LOEB: Beebe **DATE:** 11/05/80

REVIEWED BY NRR DL: **DATE:** 11/06/80

FORT ST. VRAIN

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-267
2. Utility Contact: J. W. GAHM (303) 785-2253
3. Licensed Thermal Power (MWt): 842
4. Nameplate Rating (Gross MWe): 403 x 0.85 = 343
5. Design Electrical Rating (Net MWe): 330
6. Maximum Dependable Capacity (Gross MWe): 342
7. Maximum Dependable Capacity (Net MWe): 330
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): 231
10. Reasons for Restrictions, If Any: NRC RESTRICT. 70% PEND. RESOLUTION OF TEMP. FLUCTUATIONS

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	6,575.0	10,992.0
12. Hours Reactor Was Critical	74.9	4,418.0	6,950.8
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	.0	2,974.9	3,957.1
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,100	1,351,215	1,829,170
17. Gross Electrical Energy (MWH)	0	429,129	569,925
18. Net Electrical Energy (MWH)	0	393,379	516,963
19. Unit Service Factor	.0	45.2	36.0
20. Unit Availability Factor	.0	45.2	36.0
21. Unit Capacity Factor (MDC Net)	.0	18.1	14.3
22. Unit Capacity Factor (DER Net)	.0	18.1	14.3
23. Unit Forced Outage Rate	.0	30.1	44.0

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): MAINT. & SURVEILLANCE TESTING -03/81- APPROX 3 WEEKS
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-17	08/29/80	S	720.0	B	4				THE PLANNED MAINTENANCE AND SURVEILLANCE SHUTDOWN WHICH BEGAN AUGUST 29 CONTINUED THROUGHOUT THE MONTH OF SEPTEMBER. ALTHOUGH THE REACTOR WAS TAKEN CRITICAL ON SEPTEMBER 27, THE GENERATOR REMAINED OFF-LINE.

SUMMARY

FORT ST. VRAIN REMAINED SHUTDOWN FOR PLANNED MAINTENANCE AND SURVEILLANCE TESTING.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

best copy available

GINNA

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	New York/Wayne 15 miles NE of Rochester
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	1520
5.	Design Electrical Rating (MWE Net)	470
6.	Date of Initial Criticality	November 8, 1969
7.	Date of Commercial Operation	July 1970
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Lake Ontario

Utility & Contractor Information

10.	Licensee	Rochester Gas & Electric Corporation
11.	Corporate Address	89 East Avenue Rochester, New York
12.	Corporate Contact	Leon D. White, Jr., Vice President Electric and Steam Production
13.	Architect/Engineer	Gilbert
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Section Chief	H. B. KISTER
19.	IE Resident Inspector	D. Johnson (315) 524-6935 FTS: 264-3311
20.	Licensing Project Manager	S. J. Nowicki
21.	Docket Number	50-244
22.	License Number/Date of Issue	DPR-18, 9/19/69
23.	Public Document Room	Rochester Public Library Business and Social Science Division 115 South Avenue Rochester, New York 14604

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

```
     I N S P E C T I O N   S T A T U S

     OTHER ITEMS OF INTEREST

+    FACILITY ITEMS (PLANS AND PROCEDURES):

+    A PLANNED OUTAGE WILL BEGIN ON OCTOBER 24, 1980, FOR A DURATION
     OF 33 DAYS TO COMPLETE CREVICE CLEANING CF BOTH THE "A" AND "B"
     STEAM GENERATORS.

     PLANT STATUS:

     Operating at 100% power.

* LAST IE SITE INSPECTION DATE:  9/22-26/80

+         INSPECTION REPORT NO:  50-244/80-13

     SUMMARY

     INSPECTION SUMMARY

*    NONE
```

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

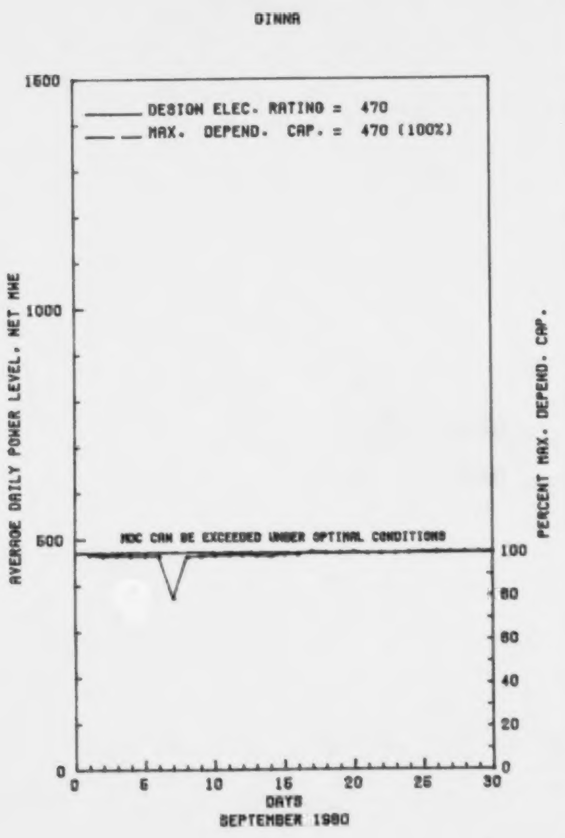
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE			

REVIEWED BY OMPA LOEB: PE Robe **DATE:** 11/4/80

REVIEWED BY NRR DL: S. J. Nowicki **DATE:** 11/5/80

GINNA

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-244
2. Utility Contact: A.E. MCNAMARA (315) 524-4446
3. Licensed Thermal Power (MWt): 1520
4. Nameplate Rating (Gross MWe): 608 x 0.85 = 517
5. Design Electrical Rating (Net MWe): 470
6. Maximum Dependable Capacity (Gross MWe): 490
7. Maximum Dependable Capacity (Net MWe): 470

CAPACITY FACTORS CALCULATED WITH A WEIGHTED AVEr AGE

8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	6,575.0	95,087.0
12. Hours Reactor Was Critical	720.0	5,274.0	72,953.8
13. Reactor Reserve Shutdown Hours	.0	.0	1,621.8
14. Hours Generator On-Line	720.0	5,235.3	71,192.1
15. Unit Reserve Shutdown Hours	.0	.0	8.5
16. Gross Thermal Energy (MWH)	1,084,296	7,766,304	36,042,106
17. Gross Electrical Energy (MWH)	351,781	2,541,794	31,227,585
18. Net Electrical Energy (MWH)	334,356	2,416,284	29,577,832
19. Unit Service Factor	100.0	79.6	74.9
20. Unit Availability Factor	100.0	79.6	74.9
21. Unit Capacity Factor (MDC Net)	98.8	78.2	68.5
22. Unit Capacity Factor (DER Net)	98.8	78.2	68.5
23. Unit Forced Outage Rate	.0	.0	9.2

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
STEAM GEN. INSPEC. & MODIFICATION - 10/25 THRU 11/26.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

II

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	NONE								

SUMMARY

GINNA OPERATED AT OR NEAR FULL POWER FOR SEPTEMBER.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

HADDAM NECK

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Connecticut/Middlesex 13 miles E of Meriden
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	1825
5.	Design Electrical Rating (MWE Net)	580
6.	Date of Initial Criticality	July 24, 1967
7.	Date of Commercial Operation	January 1, 1968
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Connecticut River

Utility & Contractor Information

10.	Licensee	Connecticut Yankee Atomic Power Co.
11.	Corporate Address	P. O. Box 270 Hartford, Connecticut
12.	Corporate Contact	D. Switzer, President
13.	Architect/Engineer	Stone & Webster
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Section Chief	R. Keimig
19.	IE Resident Inspector	T. H. Smith
20.	Licensing Project Manager	R. Caruso
21.	Docket Number	50-213
22.	License Number/Date of Issue	DPR-61, 12/27/74
23.	Public Document Room	Russell Library 119 Broad Street Middletown, Connecticut 06457

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO ENVIRONMENTAL TECHNICAL SPECIFICATION 2.4.3.1(1). ON MAY 28, 1980, A NOBLE GAS MIXTURE WAS RELEASED WHICH EXCEEDED THIS LIMIT BY A FACTOR OF 1.26.
(8007 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

+ FACILITY ITEMS (PLANS AND PROCEDURES):

+ SCHEDULED OUTAGE CONDUCTED SEPTEMBER 27-29, 1980., FOR TURBINE BALANCING AND MINOR SECONDARY PLANT MAINTENANCE.

PLANT STATUS:

+ OPERATING ROUTINELY.

+ LAST IE SITE INSPECTION DATE: 9/10-11/80

+ INSPECTION REPORT NO: 50-213/80-17

SUMMARY

INSPECTION SUMMARY

+ 50-213/80-07 - APR 28-29 - MAY 5-JUNE 7: ROUTINE, REGULAR AND BACKSHIFT INSPECTION BY THE RESIDENT INSPECTOR (48 HRS). AREAS INSPECTED INCLUDED: ACCESSIBLE AREAS OF THE TURBINE AND PRIMARY AUXILIARY BUILDINGS, THE PRIMARY CONTAINMENT, AND THE CONTROL ROOM; INVESTIGATION OF AN EVENT WHICH RESULTED IN A RADIOACTIVE GAS RELEASE FROM THE SITE; SPENT FUEL SHIPMENTS; AND, LICENSEE ACTION ON PREVIOUS INSPECTION ITEMS. ONE ITEM OF NONCOMPLIANCE WAS IDENTIFIED: FAILURE TO ADHERE TO TECHNICAL SPECIFICATION LIMITS REGARDING NOBLE GAS RELEASE RATES.

+ 50-213/80-13 - JUL 14-15: THIS INSPECTION BY ONE REGION-BASED INSPECTOR (8 HRS) WAS LIMITED TO EMERGENCY PLANNING AREAS ADDRESSED IN AN IMMEDIATE ACTION LETTER (IAL 80-20), DATED JULY 3, 1980. NO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED.

+ 50-213/80-14 - AUG 6-8: ROUTINE UNANNOUNCED INSPECTION BY A REGION-BASED INSPECTOR (38 HRS) OF THE LICENSEE'S INSERVICE INSPECTION (ISI) PROGRAM AND RELATED EXAMINATION DOCUMENTATION. NO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED.

+ 50-213/80-15 - AUG 25-28: ROUTINE UNANNOUNCED INSPECTION BY ONE REGION-BASED INSPECTOR (26 HRS) OF THE GENERAL EMPLOYEE TRAINING PROGRAM, NON-LICENSED PERSONNEL TRAINING PROGRAM AND THE LICENSED OPERATOR REQUALIFICATION PROGRAM AND AN INDEPENDENT PLANT TOUR. TWO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED: THREE LICENSED PERSONNEL WERE EXEMPTED FROM 1979 ANNUAL EXAMINATION; NO EVIDENCE AVAILABLE TO DOCUMENT REVIEW OF ALL EMERGENCY PROCEDURES BY ALL OPERATORS.

+ 50-213/80-16 (15000039/80-17) - JUL 28: VISIT TO CHEM-NUCLEAR WASTE DISPOSAL SITE, BARNWELL, S. C. FOR ANNOUNCED INSPECTIONS OF LICENSEE-SHIPPERS. EACH LICENSEE-SHIPPER VEHICLE WAS INSPECTED FOR COMPLIANCE WITH DEPARTMENT OF TRANSPORTATION (DOT) AND NUCLEAR REGULATORY COMMISSION (NRC) REGULATIONS AS FOLLOWS: (1) SHIPPING PAPER REQUIREMENTS; (2) DOT PLACARDING REQUIREMENTS; (3) RADIATION LEVELS; (4) REMOVABLE CONTAMINATION; (5) DOT MARKING AND LABELING REQUIREMENTS FOR PACKAGES; (6) DOT AND NRC REQUIREMENTS FOR EXTERNAL PACKAGE FEATURES; AND (7) PROHIBITED ARTICLES OR CONTENTS. ONE ITEM OF NONCOMPLIANCE WAS IDENTIFIED: FAILURE TO PROPERLY PLACARD THE TRANSPORTING VEHICLE.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

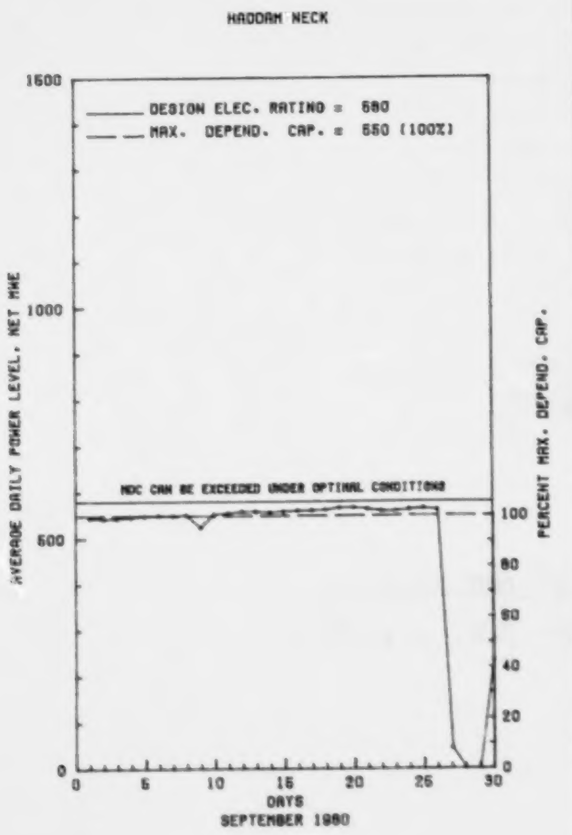
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE			

REVIEWED BY OMPA LOEB: P E Bobe **DATE:** 11/4/80

REVIEWED BY NRR DL: Ralph Caruso **DATE:** 11/5/80

HADDAM NECK

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-213
2. Utility Contact: DON ANDERSON (203) 267-2556 X274
3. Licensed Thermal Power (MWt): 1825
4. Nameplate Rating (Gross MWe): 667 x 0.9 = 600
5. Design Electrical Rating (Net MWe): 580
6. Maximum Dependable Capacity (Gross MWe): 577
7. Maximum Dependable Capacity (Net MWe): 550

CAPACITY FACTORS CALCULATED WITH A WEIGHTED AVERAGE

8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
ITEM 5-INSTALLED AN IMPROVED TURBINE ROTOR-1976
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	6,575.0	111,767.0
12. Hours Reactor Was Critical	720.0	4,578.9	95,775.6
13. Reactor Reserve Shutdown Hours	.0	27.0	1,179.6
14. Hours Generator On-Line	655.3	4,422.9	91,363.1
15. Unit Reserve Shutdown Hours	.0	.0	365.9
16. Gross Thermal Energy (MWH)	1,159,150	7,541,966	152,683,115
17. Gross Electrical Energy (MWH)	370,263	2,466,876	51,821,342
18. Net Electrical Energy (MWH)	351,748	2,339,468	49,294,231
19. Unit Service Factor	91.0	67.3	81.7
20. Unit Availability Factor	91.0	67.3	82.1
21. Unit Capacity Factor (MDC Net)	88.8	64.7	81.5
22. Unit Capacity Factor (DER Net)	84.2	61.3	75.3
23. Unit Forced Outage Rate	.0	.7	7.0

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-6	09/27/80	S	64.7	B	1		ZZ	ZZZZZZ	TURBINE BALANCE.

SUMMARY

HADDAM NECK (CONNECTICUT YANKEE) OPERATED AT NEAR FULL POWER FOR THE MONTH, WITH ONE OUTAGE ON THE 27TH TO BALANCE THE TURBINE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuation
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source

NRC POWER RESTRICTIONS: NONE

HATCH 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Georgia/Appling 11 miles N of Baxley
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2436
5.	Design Electrical Rating (MWE Net)	777
6.	Date of Initial Criticality	September 12, 1974
7.	Date of Commercial Operation	December 31, 1975
8.	Condenser Cooling Method	Once Through With Forced Draft Cooling Towers
9.	Condenser Cooling Water Source	Altamaha River

Utility & Contractor Information

10.	Licensee	Georgia Power Company
11.	Corporate Address	270 Peachtree Street, NE Atlanta, Georgia 30303
12.	Corporate Contact	J. H. Miller Executive Vice President
13.	Architect/Engineer	Southern Services, Inc.
14.	Nuclear Steam Supply System	GE
15.	Constructor	Georgia Power Company

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. Lewis
18.	IE Section Chief	H. C. Dance
19.	IE Resident Inspector	R. F. Rogers (912) 367-9881
20.	Licensing Project Manager	D. Verrelli
21.	Docket Number	50-321
22.	License Number/Date of Issue	DPR-57, 8/6/74
23.	Public Document Room	Appling County Public Library Parker Street Baxley, Georgia 31513

INSPECTION STATUS

ENFORCEMENT STATUS

AS REQUIRED BY 10 CFR 50.59 STATES, IN PART, THAT THE HOLDER OF A LICENSE AUTHORIZING OPERATION OF A PRODUCTION OR UTILIZATION FACILITY MAY MAKE CHANGES IN THE FACILITY AS DESCRIBED IN THE SAFETY ANALYSIS REPORT, WITHOUT PRIOR COMMISSION APPROVAL, UNLESS THE PROPOSED CHANGE INVOLVES A CHANGE IN THE TECHNICAL SPECIFICATIONS INCORPORATED IN THE LICENSE OR AN UNREVIEWED SAFETY QUESTION. THE LICENSEE SHALL MAINTAIN RECORDS OF CHANGES IN THE FACILITY WHICH SHALL INCLUDE A WRITTEN SAFETY EVALUATION WHICH PROVIDES THE BASIS FOR THE DETERMINATION THAT THE CHANGE DOES NOT INVOLVE AN UNREVIEWED SAFETY QUESTION. CONTRARY TO THE ABOVE, LEAD SHIELDING WAS PLACED ON AT LEAST TWO SECTIONS OF RESIDUAL HEAT REMOVAL SYSTEM PIPING LOCATED IN THE NORTHEAST DIAGONAL OF THE UNIT 1 REACTOR BUILDING WITHOUT PERFORMING AN SAFETY EVALUATION TO DETERMINE THAT THE INSTALLATION DID NOT INVOLVE AN UNREVIEWED SAFETY QUESTION.
(8027 INFRACTION)

AS REQUIRED BY TECHNICAL SPECIFICATION 6.8.1, WRITTEN PROCEDURES SHALL BE ESTABLISHED, IMPLEMENTED AND MAINTAINED COVERING THE APPLICABLE PROCEDURES RECOMMENDED IN APPENDIX 'A' OF REGULATORY GUIDE 1.33. REGULATORY GUIDE 1.33, APPENDIX A STATES IN PART THAT PROCEDURES SHOULD BE PROVIDED WHICH CONTROL RADIOACTIVITY AND LIMITS PERSONNEL RADIATION EXPOSURE. CONTRARY TO HNP-8009, A WORKER, WHO WAS NOT A MEMBER OF THE HEALTH PHYSICS STAFF, WAS OBSERVED ON JUNE 24, 1980 LEAVING A HAND AND FOOT MONITOR AFTER IT ALARMED INDICATED HIS HAND AND FEET WERE CONTAMINATED. THE WORKER PROCEEDED TO DECON HIS HAND AND SHOES WITHOUT NOTIFYING HEALTH PHYSICS. ON JUNE 23, 1980 A CHEMISTRY TECHNICIAN WAS OBSERVED LEAVING THE HAND AND FOOT MONITOR AFTER IT ALARMED INDICATING HIS HANDS WERE CONTAMINATED. THE TECHNICIAN PROCEDED TO THE CHEMISTRY LAB. CONTRARY TO HNP 8013, THE RESULTS OF AT LEAST TWELVE PARTICULATE FILTERS OR CHARCOAL CARTRIDGES ANALYZED BETWEEN APRIL 1, 1980 AND JUNE 19, 1980 EXCEEDED 1X10-9 μCi/ML FOR ALL RADIOACTIVITY AND NO ANALYSIS WAS PERFORMED TO IDENTIFY EACH RADIONUCLIDE PRESENT. CONTRARY TO HNP 8005, A WORKER WAS OBSERVED ON JUNE 24, 1980 TAKING A BAG OF TOOLS OUT OF THE RADIATION CONTROL AREA WITHOUT A SURVEY AND UNCONDITIONED RELEASE BY HEALTH PHYSICS. CONTRARY TO HNP 8010, ON SEVERAL OCCASIONS BETWEEN JUNE 16-27, 1980 THE INSPECTORS OBSERVED FULL-FACE RESPIRATORS DISCARDED ON THE FLOOR AT THE EXIT POINT FOR WORK AREAS AND SURVEYS FOR SMEARABLE CONTAMINATION ARE NOT PERFORMED ON RESPIRATORS FOLLOWING CLEANING AND PRIOR TO REUSE. CONTRARY TO HNP 8012, THE RESULTS OF CONTAMINATION SURVEY PERFORMED AT 2:20 P.M. AND 3:30 P.M. ON JUNE 18, 1980 WERE NOT DOCUMENTED.
(8027 INFRACTION)

CONTRARY TO 10 CFR 50, APPENDIX B, CRITERION I AND XVI AND SECTIONS 17.2.1.1.3 AND 17.2.18 OF THE ACCEPTED QA PROGRAM, THE QUALITY ASSURANCE DEPARTMENT DOES NOT EFFECTIVELY ASSURE CONFORMANCE TO QUALITY STANDARDS IN THAT IT DOES NOT ASSURE PROMPT CORRECTION OF CONDITIONS ADVERSE TO QUALITY. SPECIFICALLY, BETWEEN OCTOBER 27, 1978 AND APRIL 11, 1980, THE NUCLEAR REGULATORY COMMISSION HAS ISSUED FIVE CITATIONS FOR FAILURE TO TAKE CORRECTIVE ACTIONS BY THE MECHANISMS DEFINED FOR CORRECTIVE ACTION WITHIN THE QA PROGRAM (REPORTS 50-366/78-44; 50-321/79-07, 50-366/79-08; 50-366/79-09/ 50-321/79-11; 50-366/79-15; AND 50-321, 50-366/80-17). FURTHER, OF 119 QA PROGRAM AFR'S WRITTEN IN 1978 AND 135 WRITTEN IN 1979, 9 ITEMS FROM 1978 AND 35 ITEMS FROM 1979 REMAIN UNCORRECTED AS OF JULY 11, 1980. IN ADDITION, OF 45 AFR'S WRITTEN IN 1980 WHICH REQUIRED COMPLETION OF CORRECTIVE ACTION ON OR BEFORE MARCH 13, 1980, 12 REMAIN UNCORRECTED AS OF JULY 11, 1980 (I.E., OVERDUE BY 120 DAYS OR MORE).
(8028 INFRACTION)

CONTRARY TO 10 CFR 50, APPENDIX B, CRITERION II, AND SECTION 17.2 OF THE ACCEPTED QA PROGRAM, THE STATUS AND ADEQUACY OF THE QA PROGRAM WAS NOT REGULARLY REVIEWED IN THAT OF THE 45 PROCEDURES IN THE QA DEPARTMENT PROCEDURE MANUAL, 23 HAD NOT BEEN REVIEWED FOR PERIODS IN EXCESS OF TWO YEARS. TWO PROCEDURES WERE LAST REVIEWED IN 1975; SEVEN PROCEDURES WERE LAST REVIEWED IN 1976; THREE PROCEDURES WERE LAST REVIEWED IN 1977; AND ELEVEN PROCEDURES WERE LAST REVIEWED IN THE FIRST HALF OF 1978. SINCE REVIEW OF PROVISIONS IS NOT DOCUMENTED, THE LATEST REVISION DATE WAS USED TO DETERMINE THE DATE OF LAST REVIEW. (A REVISION IS CONSIDERED A REVIEW UNDER PARAGRAPH 5.2.15 OF ANSI N18-1976).
(8028 DEFICIENCY)

REPORTS RECEIVED FPOM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-084/ 03L-0	08/07/80	08/26/80	RESIDUAL HEAT REMOVAL SERVICE WATER FLOW INDICATIONS IRRATIC.
80-088/ 03L-0	07/25/80	08/15/80	HIGH PRESSURE COOLANT INJECTION FAILED OVERSPEED TEST.
80-089/ 03L-0	07/26/80	08/19/80	DRYWELL AND TORUS TEMPERATURE RECORDER INOPERABLE.
80-090/ 03L-0	07/31/80	08/19/80	DRYWELL-TORUS HYDROGEN-OXYGEN INOPERABLE.
80-092/ 01T	07/31/80	08/29/80	HIGH AMBIENT TEMPERATURE INVERTER ROOM.
80-093/ 03L-0	07/31/80	08/19/80	TORUS WATER LEVEL RECORDER INOPERABLE.
80-094/ 03L-0	07/31/80	08/19/80	OTTOASHYDROGEN SAMPLE PUMP INOPERABLE.
80-097/ 03L-0	08/15/80	09/04/80	REACTOR BUILDING VENT RADIATION MONITOR SAMPLE PUMP FAILED.
80-098/ 03L-0	08/16/80	09/02/80	TORUS-DRYWELL VACUUM BREAKER INDICATOR LIGHTS INOPERABLE.
80-099/ 03L-0	08/17/80	08/28/80	DOOR TO HIGH RADIATION AREA UNLOCKED.

==

CONTRARY TO 10 CFR 50, APPENDIX B, CRITERION IV, AND SECTION 17.2 OF THE ACCEPTED QA PROGRAM, THE LICENSEE HAS NOT ESTABLISHED MEASURES TO ASSURE THAT REQUIREMENTS ARE INCLUDED IN PROCUREMENT DOCUMENTS IN THAT THERE ARE NO PROVISIONS FOR CONTROLLING CHANGES MADE TO PURCHASE REQUISITIONS. THE INSPECTOR DETERMINED BY INTERVIEW WITH RESPONSIBLE MANAGERS, THAT CHANGES TO PURCHASE REQUISITIONS ARE MADE AFTER THESE DOCUMENTS HAVE BEEN TRANSMITTED TO THE COMPANY OFFICES FROM THE PLANT SITE AND WITHOUT THE SAME DEGREE OF CONTROL UTILIZED IN THE DOCUMENTS' PREPARATION.
(8028 DEFICIENCY)

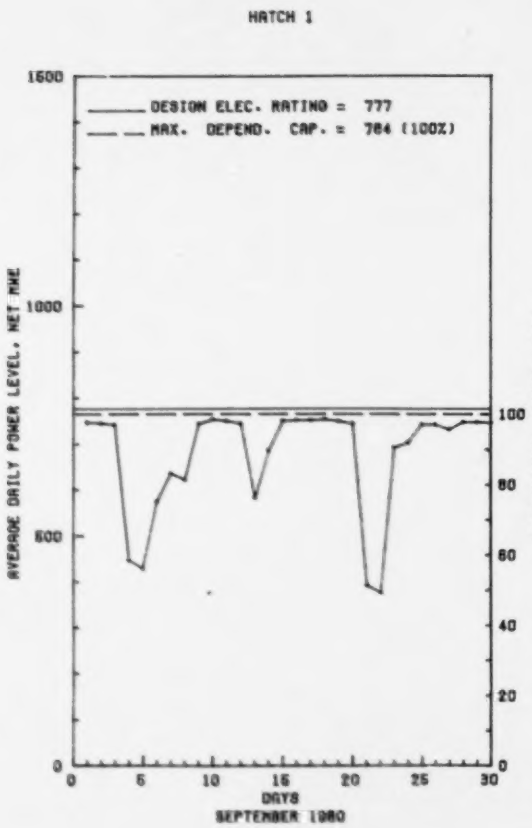
SEE CONTINUATION PAGE H-5A.

REVIEWED BY OMPA LOEB: **DATE:** 11-4-80

REVIEWED BY NRR DL: **DATE:** 11-6-80

HATCH 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-321
2. Utility Contact: P. R. UNDERWOOD (912) 367-7781
3. Licensed Thermal Power (MWt): 2436
4. Nameplate Rating (Gross MWe): 1000 x 0.85 = 850
5. Design Electrical Rating (Net MWe): 777
6. Maximum Dependable Capacity (Gross MWe): 797
7. Maximum Dependable Capacity (Net MWe): 764
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	6,575.0	41,639.0
12. Hours Reactor Was Critical	703.5	5,593.3	31,391.3
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	693.0	5,153.1	29,421.5
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,575,956	11,588,807	61,941,842
17. Gross Electrical Energy (MWH)	509,060	3,784,830	20,055,770
18. Net Electrical Energy (MWH)	487,132	3,606,894	19,068,827
19. Unit Service Factor	96.3	78.4	70.7
20. Unit Availability Factor	96.3	78.4	70.7
21. Unit Capacity Factor (MDC Net)	88.6	71.8	59.9
22. Unit Capacity Factor (DER Net)	87.1	70.6	58.9
23. Unit Forced Outage Rate	3.8	21.6	17.1

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
REFUELING AND TORUS WORK, 03/01/81 - 13 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-46	09/04/80	F	11.2	A	3		HH	HTEXCH	RX SCRAM DUE TO FALSE HIGH MSR LEVEL CAUSED BY LS BEING GROUNDED.
80-47	09/04/80	F	53.2	A	5		HH	HTEXCH	UNIT ON STARTUP RAMP FROM ABOVE SHUTDOWN, #80-46.
80-48	09/07/80	F	12.9	D	5		RB	CONROD	REDUCED LOAD TO PERFORM ROD PATTERN ADJUSTMENT.
80-49	09/12/80	S	21.6	D	5		RB	CONROD	REDUCED LOAD TO PERFORM TURBINE TEST AND ROD PATTERN ADJUSTMENT.
80-50	09/21/80	F	15.8	A	3		ZZ	ZZZZZZ	RX SCRAM DUE TO TRANSFERRING OF RPS BUS "B" FROM ALT. TO NORMAL AND A SPURIOUS TRIP ON RPS "A".
80-51	09/21/80	F	14.0	A	5		ZZ	ZZZZZZ	UNIT ON STARTUP RAMP FROM ABOVE SHUTDOWN, #80-50.
80-52	09/23/80	F	4.0	D	5		RB	CONROD	REDUCED LOAD TO PERFORM A ROD PATTERN ADJUSTMENT.
80-53	09/27/80	F	1.0	B	5		WA	FILTER	REDUCED LOAD DUE TO DEBRIS STOPPING UP PSW STRAINERS.

SUMMARY

HATCH 1 OPERATED AT ABOUT 88% POWER FOR SEPTEMBER, WITH NUMEROUS OUTAGES AND REDUCTIONS LISTED IN DETAIL ABOVE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

CONTRARY TO 10 CFR 50, APPENDIX B, CRITERION V, AND SECTION 17.2.5 OF THE ACCEPTED QA PROGRAM, THE PROCEDURE COVERING THE QUALITY CONTROL (QC) INSPECTION ACTIVITY (HNP-821, "QUALITY CONTROL WORK INSPECTION," REVISION 5, DATED 9/78) DOES NOT CONTAIN EITHER QUALITATIVE OR QUANTITATIVE ACCEPTANCE CRITERIA FOR DETERMINING WHEN THE INSPECTION ACTIVITY MUST BE ACCOMPLISHED. THE PROCEDURE STATES THAT A QUALITY CONTROL SPECIALIST REVIEWS MAINTENANCE REQUESTS (MR'S) AND THEN DETERMINES IF AN INSPECTION IS REQUIRED. THE DECISIION IS TO BE MADE "IF THE WORK INVOLVED IS SIGNIFICANT ENOUGH TO WARRANT QC INSPECTION." THE INSPECTOR FOUND NO QC INSPECTION PERFORMED ON ANY MR'S ASSOCIATED WITH INSTRUMENTATION AND CONTROL WORK (ONE CASE OF IMPROPER ALTERNATE INSPECTION WAS CONDUCTED) AND NO INVOLVEMENT IN NON-MR AREAS SUCH AS OPERATIONS AND HEALTH PHYSICS.
(8028 INFRACTION)

CONTRARY TO 10 CFR 50, APPENDIX B, CRITERION V, AND SECTION 17.2.5 OF THE ACCEPTED QA PROGRAM, ACTIVITIES AFFECTING QUALITY WERE NOT ACCOMPLISHED IN ACCORDANCE WITH PROCEDURE QA-05-06 IN THAT: (1) FOR 7 AUDIT FINDING REPORTS (AFR'S) WRITTEN IN THE LAST QUARTER OF 1979 FOR WHICH REQUIRED RESPONSES WERE NOT RECEIVED WITHIN THE 30 DAY PERIOD, THE AVERAGE PERIOD OVERDUE WAS 50.28 DAYS; AND, (2) FOR 5 AFR'S WRITTEN IN THE FIRST HALF OF 1980 FOR WHICH REQUIRED RESPONSES WERE NOT RECEIVED WITHIN 30 DAY PERIOD, 4 HAD NOT BEEN RECEIVED AS OF JULY 11, 1980 AND THE AVERAGE PERIOD OVERDUE AS OF THAT DATE WAS 87.5 DAYS.
(8028 INFRACTION)

CONTRARY TO 10 CFR 50, APPENDIX B, CRITERION VI AND SECTION 17.2.6.1 OF THE ACCEPTED QA PROGRAM, AS OF JULY 16, 1980, MEASURES HAD NOT BEEN ESTABLISHED FOR THE CONTROL OF VENDOR TECHNICAL MANUALS, AND CHANGES THERETO, REFERENCED IN SAFETY-RELATED PLANT PROCEDURES AND NECESSARY FOR CARRYING OUT THE ACTIVITIES PRESCRIBED THEREIN. SPECIFIC EXAMPLES INCLUDE BUT ARE NOT LIMITED TO: 1. CURRENT REVISIONS WERE NOT ENTERED INTO THE CONTROL ROOM, TEST SHOP AND MAINTENANCE SHOP COPIES OF MANUAL GEK 9690, VOLUME X, PART 3; 2. CURRENT REVISIONS WERE NOT ENTERED INTO THE MAINTENANCE SHOP COPY OF THE HIGH PRESSURE COOLANT INJECTION PUMP, INSTRUMENT AND OPERATIONS INSTRUCTIONS MANUAL; AND, 3. CURRENT REVISIONS WERE NOT ENTERED INTO TWO OF FOUR MASTER FILE COPIES OF THE MAIN STEAM RELIEF VALVE TECHNICAL MANUAL.
(8028 INFRACTION)

CONTRARY TO 10 CFR 50, APPENDIX B, CRITERION X, AND SECTION 17.2.10 OF THE ACCEPTED QA PROGRAM, A PROGRAM FOR INSPECTION OF ACTIVITIES AFFECTING QUALITY WAS NOT EXECUTED BY INDIVIDUALS OTHER THAN THOSE PERFORMING THE WORK ACTIVITY BEING INSPECTED AS DEFINED IN THE LICENSEE'S ACCEPTED QA PROGRAM; THE LICENSEE HAS NOT ESTABLISHED AN INSPECTION PROGRAM IN THAT PORTIONS OF THE QA PROGRAM WHICH ALLOW PERSONNEL WITHIN A DEPARTMENT TO PERFORM SURVEILLANCE INSPECTION OF WORK WITHIN THAT DEPARTMENT (THE ALTERNATE INSPECTION PROGRAM) HAVE NOT BEEN DOCUMENTED BY WRITTEN PROCEDURES; AND THE LICENSEE HAS NOT ESTABLISHED AN INSPECTION PROGRAM IN ACCORDANCE WITH THE ACCEPTED QA PROGRAM IN THAT THERE ARE NO PROVISIONS TO CERTIFY PERSONS PERFORMING SHIPPING DAMAGE INSPECTIONS. BASED ON INTERVIEWS AND OBSERVATIONS, THE INSPECTOR DID VERIFY THAT THOSE PERSONS ASSIGNED THIS INSPECTION ACTIVITY WERE QUALIFIED TO PERFORM THE FUNCTION.
(8028 INFRACTION)

CONTRARY TO 10 CFR 50, APPENDIX B, CRITERION XIII, AND SECTION 17.2 OF THE ACCEPTED QA PROGRAM, MEASURES WERE NOT ESTABLISHED TO CONTROL THE STORAGE OF MATERIALS AND EQUIPMENT TO PREVENT DAMAGE TO WELD END PREPARATIONS IN THAT: 1. THE LICENSEE HAD NO PROGRAM TO INSTALL CAPS OR PLUGS AS REQUIRED BY THE PROGRAM'S COMMITMENT TO ANSI N45.2.2; AND, 2. THE INSPECTOR FOUND NUMEROUS STAINLESS STEEL ELBOW, WITH WELD END PREPARATIONS, PURCHASED UNDER ORDER PEN2-6442 THAT WERE STORED WITHOUT THE REQUIRED CAPS OR PLUGS. NONE OF THE STORED ELBOWS HAS ANY OBSERVABLE DAMAGE TO THE WELD END PREPARATIONS.
(8028 DEFICIENCY)

CONTRARY TO 10 CFR 50, APPENDIX B, CRITERION XVII, AND SECTION 17.2.17 OF THE ACCEPTED QA PROGRAM, RECORDS OF INSPECTIONS WERE NOT MAINTAINED AND REQUIREMENTS CONCERNING RECORD LOCATION WERE NOT ESTABLISHED IN THAT: 1. RECORDS OF SHIPPING DAMAGE INSPECTIONS ARE NOT MAINTAINED, AND 2. UNIT 1 AND 2 RECORDS ARE STORED IN THE GEORGIA POWER COMPANY BLOCK HOUSE ON SITE AND THIS STRUCTURE IS NOT DESCRIBED AS A RECORD STORAGE LOCATION IN THE LCIENSEE'S PROCEDURE HNP-820, "PLANT RECORDS MANAGEMENT", REVISION 10 DATED 8/79.
(8028 DEFICIENCY)

CONTRARY TO 10 CFR 50, APPENDIX B, CRITERION XVIII, AND SECTION 17.2.18 OF THE ACCEPTED QA PROGRAM, REAUDITS OF DEFICIENT AREAS WERE NOT CONDUCTED WHERE INDICATED IN THAT ITEMS IDENTIFIED DURING THREE SPECIFIC AUDITS HAD NOT BEEN REAUDITED NOR WAS THE CURRENT STATUS OF CORRECTION KNOWN BY THE AUDITING ORGANIZATION AS OF JULY 24, 1980. SPECIFICALLY, THE AUDIT OF POWER SUPPLY ENGINEERING AND SERVICES CONDUCTED SEPTEMBER 5-7, 1978 IDENTIFIED 7 DEFICIENT AREAS WHICH HAD NOT BEEN REAUDITED NOR WAS THEIR STATUS KNOWN; AND THE AUDIT OF ENVIRONMENTAL PROGRAMS CONDUCTED OCTOBER 17-18, 1979 IDENTIFIED 2 DEFICIENT AREAS ONE OF WHICH WAS CLOSED ON FEBRUARY 5, 1980, BUT THE OTHER HAD NOT BEEN REAUDITED NOR WAS ITS STATUS KNOWN; AND, THE AUDIT OF "Q" MOTOR REPAIR PROGRAM CONDUCTED JANUARY 24, 1980, IDENTIFIED 2 DEFICIENT AREAS WHICH HAD NOT BEEN REAUDITED, WHICH HAD NO SPECIFIED COMPLETION DATE, AND THE STATUS OF CORRECTIONS WAS NOT KNOWN.
(8028 INFRACTION)

CONTRARY TO TECHNICAL SPECIFICATION 6.8.1, PROCEDURE HNP-821 WAS NOT IMPLEMENTED IN THAT THE ALTERNATE INSPECTORS VERIFYING ACTIVITIES ASSOCIATED WITH THE REPLACEMENT AND REPAIR OF 270 VALVES ON THE HYDRAULIC CONTROL UNITS OF UNIT 1 IN JULY OF 1979 WERE NOT DESIGNATED BY THE SITE QUALITY CONTROL SUPERVISOR NOR WERE THE INSPECTION RESULTS DOCUMENTED ON THE SPECIFIED FORMS; AND PROCEDURE HNP-8 WAS NOT IMPLEMENTED IN THAT FOR 4 OF 25 MAINTENANCE REQUESTS REVIEWED (1-80-3398, 1-80-3619, 1-80-3414 AND 1-80-3511) ITEMS WERE REPLACED AND NO INVENTORY MATERIAL REQUEST WAS USED AND THE PARTS WERE NOT IDENTIFIED ON THE MAINTENANCE REQUEST.
(8028 INFRACTION)

CONTRARY TO TECHNICAL SPECIFICATION 6.8.3, PROCEDURES REQUIRED BY TECHNICAL SPECIFICATION 6.8.1 ARE NOT PERIODICALLY REVIEWED AS SET FORTH IN HNP-814 IN THAT THE 700 PROCEDURES SPECIFIED BY THE RECORDS DEPARTMENT TO BE REVIEWED IN THE MONTHS OF FEBRUARY, MARCH, APRIL MAY AND JUNE OF 1980, 540 WERE NOT REVIEWED IN THE MONTH SPECIFIED AND 473 REMAINED UNREVIEWED AS OF JULY 10, 1980.
(8028 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

PLANT IS OPERATING AT FULL POWER.

◆ LAST IE SITE INSPECTION DATE: SEPTEMBER 12, 1980

◆ INSPECTION REPORT NO: 50-321/80-39

SUMMARY

INSPECTION SUMMARY

◆ INSPECTION JUNE 16-27 (80-27): THIS SPECIAL, ANNOUNCED INSPECTION INVOLVED 181 INSPECTOR-HOURS ON-SITE IN THE AREA OF HEALTH PHYSICS APPRAISAL INCLUDING RADIATION PROTECTION ORGANIZATION AND MANAGEMENT, PERSONNEL SELECTION, QUALIFICATIONS AND TRAINING, EXPOSURE CONTROLS, ALARA PROGRAM, RADIOACTIVE WASTE MANAGEMENT, FACILITIES AND EQUIPMENT, AND EMERGENCY RESPONSE CAPABILITIES. OF THE SEVEN AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN SIX AREAS; TWO ITEMS OF NONCOMPLIANCE WERE FOUND IN ONE AREA (INFRACTION - FAILURE TO FOLLOW WRITTEN PROCEDURES; INFRACTION - FAILURE TO PERFORM A SAFETY ANALYSIS PRIOR TO MAKING CHANGES TO THE FACILITY).

INSPECTION JULY 31 (80-32): INSPECTED CHEM-NUCLEAR WASTE DISPOSAL SITE, BARNWELL, SOUTH CAROLINA FOR UNANNOUNCED INSPECTIONS OF LICENSEE-SHIPPERS. EACH LICENSEE-SHIPPER VEHICLE WAS INSPECTED FOR COMPLIANCE WITH DEPARTMENT OF TRANSPORTATION (DOT) AND NUCLEAR REGULATORY COMMISSION (NRC) REGULATIONS AS FOLLOWS: (1) SHIPPING PAPER REQUIREMENTS; (2) DOT PLACARDING REQUIREMENTS; (3) RADIATION LEVELS; (4) REMOVABLE CONTAMINATION, (5) DOT MARKING AND LABELING REQUIREMENTS FOR PACKAGES; (6) DOT AND NRC REQUIREMENTS FOR EXTERNAL PACKAGE FEATURES, AND (7) PROHIBITED ARTICLES OR CONTENTS. OF THE SEVEN AREAS INSPECTED ONE ITEM OF NONCOMPLIANCE WAS OBSERVED IN ONE SHIPMENT (SEVERITY LEVEL IV - FAILURE TO PROPERLY PLACARD VEHICLE).

INSPECTION JULY 19 - AUGUST 15 (80-35): THIS INSPECTION INVOLVED 38 INSPECTOR-HOURS ON SITE IN THE AREAS OF TECHNICAL SPECIFICATION COMPLIANCE, REPORTABLE OCCURRENCES, HOUSEKEEPING, OPERATOR PERFORMANCE, OVERALL PLANT OPERATIONS, QUALITY ASSURANCE PRACTICES, STATION AND CORPORATE MANAGEMENT PRACTICES, CORRECTIVE AND PREVENTATIVE MAINTENANCE ACTIVITIES, SITE SECURITY PROCEDURES, RADIATION CONTROL ACTIVITIES AND SURVEILLANCE ACTIVITIES.

OF THE ELEVEN AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN TEN AREAS. ONE ITEM OF NONCOMPLIANCE WAS FOUND IN ONE AREA (INFRACTION - REACTOR STARTUPS WITH INOPERATIVE SCRAM VOLUME LEVEL SWITCHES).

INSPECTION AUGUST 16 - SEPTEMBER 12 (80-39): THIS INSPECTION INVOLVED 44 INSPECTOR-HOUR ON SITE IN THE AREAS OF TECHNICAL SPECIFICATION COMPLIANCE, REPORTABLE OCCURRENCES, HOUSEKEEPING, OPERATOR PERFORMANCE, OVERALL PLANT OPERATIONS, QUALITY ASSURANCE PRACTICES, STATION AND CORPORATE MANAGEMENT PRACTICES, CORRECTIVE AND PREVENTATIVE MAINTENANCE ACTIVITIES, SITE SECURITY PROCEDURES, RADIATION CONTROL ACTIVITIES AND SURVEILLANCE ACTIVITIES. OF THE ELEVEN AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

HATCH 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Georgia/Appling 11 miles N of Baxley
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2436
5.	Design Electrical Rating (MWE Net)	784
6.	Date of Initial Criticality	July 4, 1978
7.	Date of Commercial Operation	September 5, 1979
8.	Condenser Cooling Method	Cooling Towers
9.	Condenser Cooling Water Source	Altamaha River

Utility & Contractor Information

10.	Licensee	Georgia Power Company
11.	Corporate Address	270 Peach Street, NE Atlanta, Georgia 30303
12.	Corporate Contact	J. H. Miller Executive Vice President
13.	Architect/Engineer	Southern Services, Inc./Bechtel
14.	Nuclear Steam Supply System	GE
15.	Constructor	Georgia Power Company

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. Lewis
18.	IE Section Chief	H. C. Dance
19.	IE Resident Inspector	R. F. Rogers (912) 367-9881
20.	Licensing Project Manager	D. Verrelli
21.	Docket Number	50-366
22.	License Number/Date of Issue	NPF-5/June 13, 1978
23.	Public Document Room	Appling County Public Library Parker Street Baxley, Georgia 31513

INSPECTION STATUS

ENFORCEMENT STATUS

AS REQUIRED BY 10 CFR 50.59 STATES, IN PART, THAT THE HOLDER OF A LICENSE AUTHORIZING OPERATION OF A PRODUCTION OR UTILIZATION FACILITY MAY MAKE CHANGES IN THE FACILITY AS DESCRIBED IN THE SAFETY ANALYSIS REPORT, WITHOUT PRIOR COMMISSION APPROVAL, UNLESS THE PROPOSED CHANGE INVOLVES A CHANGE IN THE TECHNICAL SPECIFICATIONS INCORPORATED IN THE LICENSE OR AN UNREVIEWED SAFETY QUESTION. THE LICENSEE SHALL MAINTAIN RECORDS OF CHANGES IN THE FACILITY WHICH SHALL INCLUDE A WRITTEN SAFETY EVALUATION WHICH PROVIDES THE BASIS FOR THE DETERMINATION THAT THE CHANGE DOES NOT INVOLVE AN UNREVIEWED SAFETY QUESTION. CONTRARY TO THE ABOVE, LEAD SHIELDING WAS PLACED ON AT LEAST TWO SECTIONS OF RESIDUAL HEAT REMOVAL SYSTEM PIPING LOCATED IN THE NORTHEAST DIAGONAL OF THE UNIT 1 REACTOR BUILDING WITHOUT PERFORMING A SAFETY EVALUATION TO DETERMINE THAT THE INSTALLATION DID NOT INVOLVE AN UNREVIEWED SAFETY QUESTION.
(8027 INFRACTION)

AS REQUIRED BY TECHNICAL SPECIFICATION 6.8.1, WRITTEN PROCEDURES SHALL BE ESTABLISHED, IMPLEMENTED AND MAINTAINED COVERING THE APPLICABLE PROCEDURES RECOMMENDED IN APPENDIX "A" OF REGULATORY GUIDE 1.33. REGULATORY GUIDE 1.33, APPENDIX A STATES IN PART THAT PROCEDURES SHOULD BE PROVIDED WHICH CONTROL RADIOACTIVITY AND LIMITS PERSONNEL RADIATION EXPOSURE. CONTRARY TO HNP-8003, A WORKER, WHO WAS NOT A MEMBER OF THE HEALTH PHYSICS STAFF, WAS OBSERVED ON JUNE 24, 1980 LEAVING A HAND AND FOOT MONITOR AFTER IT ALARMED INDICATED HIS HAND AND FEET WERE CONTAMINATED. THE WORKER PROCEEDED TO DECON HIS HANDS AND SHOES WITHOUT NOTIFYING HEALTH PHYSICS. ON JUNE 23, 1980 A CHEMISTRY TECHNICIAN WAS OBSERVED LEAVING THE HAND AND FOOT MONITOR AFTER IT ALARMED INDICATING HIS HANDS WERE CONTAMINATED. THE TECHNICIAN PROCEDED TO THE CHEMISTRY LAB. CONTRARY TO HNP 8013, THE RESULTS OF AT LEAST TWELVE PARTICULATE FILTERS OR CHARCOAL CARTRIDGES ANALYZED BETWEEN APRIL 1, 1980 AND JUNE 19, 1980 EXCEEDED 1X10-9 UCI/ML FOR ALL RADIOACTIVITY AND NO ANALYSIS WAS PERFORMED TO IDENTIFY EACH RADIONUCLIDE PRESENT. CONTRARY TO HNP 8035, A WORKER WAS OBSERVED ON JUNE 24, 1980 TAKING A BAG OF TOOLS OUT OF THE RADIATION CONTROL AREA WITHOUT A SURVEY AND UNCONDITIONED RELEASE BY HEALTH PHYSICS. CONTRARY TO HNP 8010, ON SEVERAL OCCASIONS BETWEEN JUNE 16-27, 1980 THE INSPECTORS OBSERVED FULL-FACE RESPIRATORS DISCARDED ON THE FLOOR AT THE EXIT POINT FOR WORK AREAS AND SURVEYS FOR SMEARABLE CONTAMINATION ARE NOT PERFORMED ON RESPIRATORS FOLLOWING CLEANING AND PRIOR TO REUSE. CONTRARY TO HNP 8012, THE RESULTS OF CONTAMINATION SURVEY PERFORMED AT 2:20 P.M. AND 3:30 P.M. ON JUNE 18, 1980 WERE NOT DOCUMENTED.
(8027 INFRACTION)

CONTRARY TO 10 CFR 50, APPENDIX B, CRITERION I AND XVI AND SECTIONS 17.2.1.1.3 AND 17.2.18 OF THE ACCEPTED QA PROGRAM, THE QUALITY ASSURANCE DEPARTMENT DOES NOT EFFECTIVELY ASSURE CONFORMANCE TO QUALITY STANDARDS IN THAT IT DOES NOT ASSURE PROMPT CORRECTION OF CONDITIONS ADVERSE TO QUALITY. SPECIFICALLY, BETWEEN OCTOBER 27, 1978 AND APRIL 11, 1980, THE NUCLEAR REGULATORY COMMISSION HAS ISSUED FIVE CITATIONS FOR FAILURE TO TAKE CORRECTIVE ACTIONS BY THE MECHANISMS DEFINED FOR CORRECTIVE ACTION WITHIN THE QA PROGRAM (REPORTS 50-366/78-44; 50-321/79-07, 50-366/79-08; 50-366/79-09/ 50-321/79-11; 50-366/79-15; AND 50-321, 50-366/80-17). FURTHER, OF 119 QA PROGRAM AFR'S WRITTEN IN 1978 AND 135 WRITTEN IN 1979, 9 ITEMS FROM 1978 AND 35 ITEMS FROM 1979 REMAIN UNCORRECTED AS OF JULY 11, 1980. IN ADDITION, OF 45 AFR'S WRITTEN IN 1980 WHICH REQUIRED COMPLETION OF CORRECTIVE ACTION ON OR BEFORE MARCH 13, 1980, 12 REMAIN UNCORRECTED AS OF JULY 11, 1980 (I.E., OVERDUE BY 120 DAYS OR MORE).
(8028 INFRACTION)

CONTRARY TO 10 CFR 50, APPENDIX B, CRITERION II, AND SECTION 17.2 OF THE ACCEPTED QA PROGRAM, THE STATUS AND ADEQUACY OF THE QA PROGRAM WAS NOT REGULARLY REVIEWED IN THAT OF THE 45 PROCEDURES IN THE QA DEPARTMENT PROCEDURE MANUAL, 23 HAD NOT BEEN REVIEWED FOR PERIODS IN EXCESS OF TWO YEARS. TWO PROCEDURES WERE LAST REVIEWED IN 1975; SEVEN PROCEDURES WERE LAST REVIEWED IN 1976; THREE PROCEDURES WERE LAST REVIEWED IN 1977; AND ELEVEN PROCEDURES WERE LAST REVIEWED IN THE FIRST HALF OF 1978. SINCE REVIEW OF PROVISIONS IS NOT DOCUMENTED, THE LATEST REVISION DATE WAS USED TO DETERMINE THE DATE OF LAST REVIEW. (A REVISION IS CONSIDERED A REVIEW UNDER PARAGRAPH 5.2.15 OF ANSI N18-1976).
(8028 DEFICIENCY)

CONTRARY TO 10 CFR 50, APPENDIX B, CRITERION IV, AND SECTION 17.2 OF THE ACCEPTED QA PROGRAM, THE LICENSEE HAS NOT ESTABLISHED MEASURES TO ASSURE THAT REQUIREMENTS ARE INCLUDED IN PROCUREMENT DOCUMENTS IN THAT THERE ARE NO PROVISIONS FOR CONTROLLING CHANGES MADE TO PURCHASE REQUISITIONS. THE INSPECTOR DETERMINED BY INTERVIEW WITH RESPONSIBLE MANAGERS, THAT CHANGES TO PURCHASE REQUISITIONS ARE MADE AFTER THESE DOCUMENTS HAVE BEEN TRANSMITTED TO THE COMPANY OFFICES FROM THE PLANT SITE AND WITHOUT THE SAME DEGREE OF CONTROL UTILIZED IN THE DOCUMENTS' PREPARATION.
(8028 DEFICIENCY)

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-106 01T	07/28/80	08/26/80	DEPOSITS FOUND ON INSULATION IN DRYWELL.
80-109 03L-0	08/09/80	08/26/80	HIGH PRESSURE COOLANT INJECTION SUCTION VALVE FAILED TO OPEN.
80-111/ 03L-0	07/24/80	08/19/80	HIGH PRESSURE COOLANT INJECTION STEAM SUPPLY VALVE SHUT ON SPURIOUS SIGNAL.
80-112/ 03L-0	07/23/80	08/19/80	HIGH PRESSURE COOLANT INJECTION SYSTEM STEAM FLOW SWITCH INOPERATIVE.
80-113/ 01T	07/26/80	08/25/80	SCRAM LEVEL SWITCHES FAILED TO FUNCTION DURING SCRAM.
80-114/ 03L-0	07/26/80	08/19/80	HIGH PRESSURE COOLANT INJECTION SYSTEM AUTO ISOLATED BELOW SET PRESSURE REQUIREMENTS.
80-115/ 03L-0	07/27/80	08/20/80	2C DIESEL GENERATOR FAILED TO TIE INTO BUS DURING TEST.
80-116/ 03L-0	08/02/80	08/19/80	TORUS TEMP. EXCEED TECHNICAL SPECIFICATION REQUIREMENTS.
80-117/ 03L-0	08/03/80	08/19/80	REACTOR CORE ISOLATION COOLING SYSTEM TURBINE RESET COULD NOT BE ACCOMPLISHED DUE TO INADEQUATE PROCEDURE.
80-118/ 03L-0	08/04/80	08/19/80	HYDROGEN-OXYGEN ANALYZER "A" DRYWELL INOPERATIVE.
80-119/ 03L-0	08/05/80	08/29/80	FIRE BARRIER SEAL CONTROL ROOM FLOOR BREACHED.
80-120 03L-0	08/11/80	08/21/80	LOSS DC POWER TO MG SET OIL PUMP.
80-121 03L-0	08/13/80	08/19/80	REACTOR PRESSURE SWITCH SETPOINT DRIFT.
80-122/ 03L-0	08/11/80	08/26/80	REACTOR PRESSURE SWITCH SETPOINT DRIFT.
80-123/ 03L-0	08/12/80	08/29/80	STEAM LINE ISOLATION VALVE FOR REACTOR CORE ISOLATION COOLING FAILED TO CLOSE DURING TEST.
80-124/ 03L-0	08/13/80	09/04/80	TORUS-DRYWELL VACUUM BREAKERS LIGHT INDICATION NOT OPERATING PROPERLY.

CONTRARY TO 10 CFR 50, APPENDIX B, CRITERION V, AND SECTION 17.2.5 OF THE ACCEPTED QA PROGRAM, THE PROCEDURE COVERING THE QUALITY CONTROL (QC) INSPECTION ACTIVITY (HNP-821, "QUALITY CONTROL WORK INSPECTION," REVISION 5, DATED 9/78) DOES NOT CONTAIN EITHER QUALITATIVE OR QUANTITATIVE ACCEPTANCE CRITERIA FOR DETERMINING WHEN THE INSPECTION ACTIVITY MUST BE ACCOMPLISHED. THE PROCEDURE STATES THAT A QUALITY CONTROL SPECIALIST REVIEWS MAINTENANCE REQUESTS (MR'S) AND THEN DETERMINES IF AN INSPECTION IS REQUIRED. THE DECISIION IS TO BE MADE "IF THE WORK INVOLVED IS SIGNIFICANT ENOUGH TO WARRANT QC INSPECTION." THE INSPECTOR FOUND NO QC INSPECTION PERFORMED ON ANY MR'S ASSOCIATED WITH INSTRUMENTATION AND CONTROL WORK (ONE CASE OF IMPROPER ALTERNATE INSPECTION WAS CONDUCTED) AND NO INVOLVEMENT IN NON-MR AREAS SUCH AS OPERATIONS AND HEALTH PHYSICS.
(8028 INFRACTION)

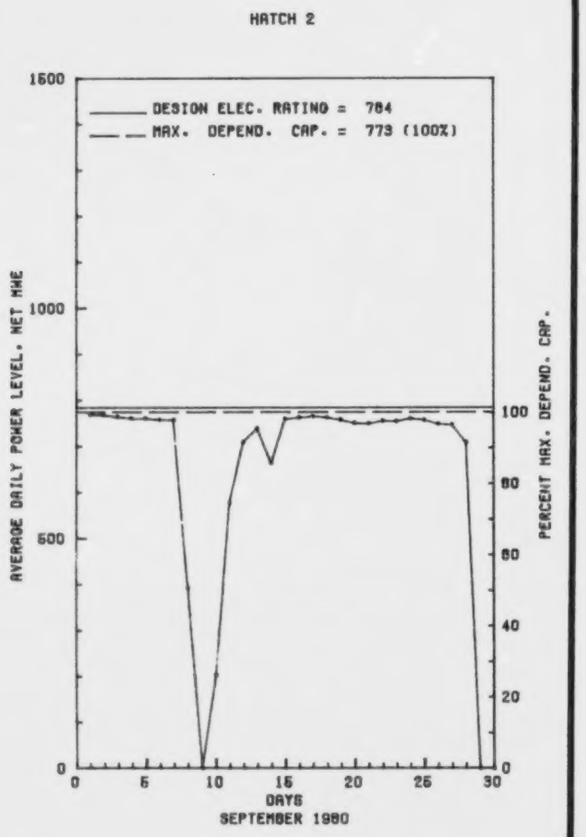
SEE CONTINUATION PAGE H-7A.

REVIEWED BY OMPA LOEB: **DATE:** 11-4-80

REVIEWED BY NRR DL: **DATE:** 11-6-80

HATCH 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-366
2. Utility Contact: P. R. UNDERWOOD (912) 367-7781
3. Licensed Thermal Power (MWt): 2436
4. Nameplate Rating (Gross MWe): 1000 x .85 = 850
5. Design Electrical Rating (Net MWe): 784
6. Maximum Dependable Capacity (Gross MWe): 806
7. Maximum Dependable Capacity (Net MWe): 773
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	6,575.0	9,408.0
12. Hours Reactor Was Critical	660.2	4,850.3	7,513.8
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	636.0	4,573.3	6,968.0
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,467,566	10,106,760	15,600,822
17. Gross Electrical Energy (MWH)	480,630	3,347,360	5,183,320
18. Net Electrical Energy (MWH)	458,908	3,189,347	4,946,484
19. Unit Service Factor	88.3	69.6	74.3
20. Unit Availability Factor	88.3	69.6	74.3
21. Unit Capacity Factor (MDC Net)	82.5	62.8	68.0
22. Unit Capacity Factor (DER Net)	81.3	61.9	67.1
23. Unit Forced Outage Rate	11.7	13.2	11.3

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
REFUELING & TORUS WORK, 11/01/80 - 13 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: 10/01/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-34	09/08/80	F	40.3	A	3		CD	VALVOP	RX AUTO SCRAM DUE TO MSIV NOT FULLY OPEN.
80-35	09/09/80	F	38.0	A	5		CD	VALVOP	UNIT ON STARTUP RAMP FROM ABOVE SHUTDOWN, #80-34.
80-36	09/13/80	S	7.0	B	5		HE	TURBIN	REDUCED LOAD IN ORDER TO PERFORM WEEKLY TURBINE TEST.
80-37	09/28/80	F	4.5	A	5	80-135	HC	HTEXCH	REDUCED LOAD DUE TO HIGH CONDUCTIVITY.
80-38	09/29/80	F	43.7	A	3		HC	HTEXCH	RX SCRAM DUE TO LOW CONDENSER VACUUM.
80-39	09/30/80	F	2.3	A	5		HC	HTEXCH	UNIT ON STARTUP RAMP FROM ABOVE SHUTDOWN #80-38.

SUMMARY

HATCH 2 OPERATED NORMALLY DURING SEPTEMBER WITH NUMEROUS OUTAGES AND REDUCTIONS LISTED IN DETAIL ABOVE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

CONTINUATION PAGE

ENFORCEMENT STATUS

CONTRARY TO 10 CFR 50, APPENDIX B, CRITERION V, AND SECTION 17.2.5 OF THE ACCEPTED QA PROGRAM, ACTIVITIES AFFECTING QUALITY WERE NOT ACCOMPLISHED IN ACCORDANCE WITH PROCEDURE QA-05-06 IN THAT: (1) FOR 7 AUDIT FINDING REPORTS (AFR'S) WRITTEN IN THE LAST QUARTER OF 1979 FOR WHICH REQUIRED RESPONSES WERE NOT RECEIVED WITHIN THE 30 DAY PERIOD, THE AVERAGE PERIOD OVERDUE WAS 50.28 DAYS; AND, (2) FOR 5 AFR'S WRITTEN IN THE FIRST HALF OF 1980 FOR WHICH REQUIRED RESPONSES WERE NOT RECEIVED WITHIN 30 DAY PERIOD, 4 HAD NOT BEEN RECEIVED AS OF JULY 11, 1980 AND THE AVERAGE PERIOD OVERDUE AS OF THAT DATE WAS 87.5 DAYS.
(8028 INFRACTION)

CONTRARY TO 10 CFR 50, APPENDIX B, CRITERION VI AND SECTION 17.2.6.1 OF THE ACCEPTED QA PROGRAM, AS OF JULY 16, 1980, MEASURES HAD NOT BEEN ESTABLISHED FOR THE CONTROL OF VENDOR TECHNICAL MANUALS, AND CHANGES THERETO, REFERENCED IN SAFETY-RELATED PLANT PROCEDURES AND NECESSARY FOR CARRYING OUT THE ACTIVITIES PRESCRIBED THEREIN. SPECIFIC EXAMPLES INCLUDE BUT ARE NOT LIMITED TO: 1. CURRENT REVISIONS WERE NOT ENTERED INTO THE CONTROL ROOM, TEST SHOP AND MAINTENANCE SHOP COPIES OF MANUAL GEK 9690, VOLUME X, PART 3; 2. CURRENT REVISIONS WERE NOT ENTERED INTO THE MAINTENANCE SHOP COPY OF THE HIGH PRESSURE COOLANT INJECTION PUMP, INSTRUMENT AND OPERATIONS INSTRUCTIONS MANUAL; AND, 3. CURRENT REVISIONS WERE NOT ENTERED INTO TWO OF FOUR MASTER FILE COPIES OF THE MAIN STEAM RELIEF VALVE TECHNICAL MANUAL.
(8028 INFRACTION)

CONTRARY TO 10 CFR 50, APPENDIX B, CRITERION X, AND SECTION 17.2.10 OF THE ACCEPTED QA PROGRAM, A PROGRAM FOR INSPECTION OF ACTIVITIES AFFECTING QUALITY WAS NOT EXECUTED BY INDIVIDUALS OTHER THAN THOSE PERFORMING THE WORK ACTIVITY BEING INSPECTED AS DEFINED IN THE LICENSEE'S ACCEPTED QA PROGRAM; THE LICENSEE HAS NOT ESTABLISHED AN INSPECTION PROGRAM IN THAT PORTIONS OF THE QA PROGRAM WHICH ALLOW PERSONNEL WITHIN A DEPARTMENT TO PERFORM SURVEILLANCE INSPECTION OF WORK WITHIN THAT DEPARTMENT (THE ALTERNATE INSPECTION PROGRAM) HAVE NOT BEEN DOCUMENTED BY WRITTEN PROCEDURES; AND THE LICENSEE HAS NOT ESTABLISHED AN INSPECTION PROGRAM IN ACCORDANCE WITH THE ACCEPTED QA PROGRAM IN THAT THERE ARE NO PROVISIONS TO CERTIFY PERSONS PERFORMING SHIPPING DAMAGE INSPECTIONS. BASED ON INTERVIEWS AND OBSERVATIONS, THE INSPECTOR DID VERIFY THAT THOSE PERSONS ASSIGNED THIS INSPECTION ACTIVITY WERE QUALIFIED TO PERFORM THE FUNCTION.
(8028 INFRACTION)

CONTRARY TO 10 CFR 50, APPENDIX B, CRITERION XIII, AND SECTION 17.2 OF THE ACCEPTED QA PROGRAM, MEASURES WERE NOT ESTABLISHED TO CONTROL THE STORAGE OF MATERIALS AND EQUIPMENT TO PREVENT DAMAGE TO WELD END PREPARATIONS IN THAT: 1. THE LICENSEE HAD NO PROGRAM TO INSTALL CAPS OR PLUGS AS REQUIRED BY THE PROGRAM'S COMMITMENT TO ANSI N45.2.2; AND, 2. THE INSPECTOR FOUND NUMEROUS STAINLESS STEEL ELBOW, WITH WELD END PREPARATIONS, PURCHASED UNDER ORDER PEH2-6442 THAT WERE STORED WITHOUT THE REQUIRED CAPS OR PLUGS. NONE OF THE STORED ELBOWS HAS ANY OBSERVABLE DAMAGE TO THE WELD END PREPARATIONS.
(8028 DEFICIENCY)

CONTRARY TO 10 CFR 50, APPENDIX B, CRITERION XVII, AND SECTION 17.2.17 OF THE ACCEPTED QA PROGRAM, RECORDS OF INSPECTIONS WERE NOT MAINTAINED AND REQUIREMENTS CONCERNING RECORD LOCATION WERE NOT ESTABLISHED IN THAT: 1. RECORDS OF SHIPPING DAMAGE INSPECTIONS ARE NOT MAINTAINED, AND 2. UNIT 1 AND 2 RECORDS ARE STORED IN THE GEORGIA POWER COMPANY BLOCK HOUSE ON SITE AND THIS STRUCTURE IS NOT DESCRIBED AS A RECORD STORAGE LOCATION IN THE LCIENSEE'S PROCEDURE HNP-820, "PLANT RECORDS MANAGEMENT", REVISION 10 DATED 8/79.
(8028 DEFICIENCY)

CONTRARY TO 10 CFR 50, APPENDIX B, CRITERION XVIII, AND SECTION 17.2.18 OF THE ACCEPTED QA PROGRAM, REAUDITS OF DEFICIENT AREAS WERE NOT CONDUCTED WHERE INDICATED IN THAT ITEMS IDENTIFIED DURING THREE SPECIFIC AUDITS HAD NOT BEEN REAUDITED NOR WAS THE CURRENT STATUS OF CORRECTION KNOWN BY THE AUDITING ORGANIZATION AS OF JULY 24, 1980. SPECIFICALLY, THE AUDIT OF POWER SUPPLY ENGINEERING AND SERVICES CONDUCTED SEPTEMBER 5-7, 1978 IDENTIFIED 7 DEFICIENT AREAS WHICH HAD NOT BEEN REAUDITED NOR WAS THEIR STATUS KNOWN; AND THE AUDIT OF ENVIRONMENTAL PROGRAMS CONDUCTED OCTOBER 17-18, 1979 IDENTIFIED 2 DEFICIENT AREAS ONE OF WHICH WAS CLOSED ON FEBRUARY 5, 1980, BUT THE OTHER HAD NOT BEEN REAUDITED NOR WAS ITS STATUS KNOWN; AND, THE AUDIT OF "Q" MOTOR REPAIR PROGRAM CONDUCTED JANUARY 24, 1980, IDENTIFIED 2 DEFICIENT AREAS WHICH HAD NOT BEEN REAUDITED, WHICH HAD NO SPECIFIED COMPLETION DATE, AND THE STATUS OF CORRECTIONS WAS NOT KNOWN.
(8028 INFRACTION)

CONTRARY TO TECHNICAL SPECIFICATION 6.8.3, PROCEDURES REQUIRED BY TECHNICAL SPECIFICATION 6.8.1 ARE NOT PERIODICALLY REVIEWED AS SET FORTH IN HNP-814 IN THAT THE 700 PROCEDURES SPECIFIED BY THE RECORDS DEPARTMENT TO BE REVIEWED IN THE MONTHS OF FEBRUARY, MARCH, APRIL MAY AND JUNE OF 1980, 540 WERE NOT REVIEWED IN THE MONTH SPECIFIED AND 473 REMAINED UNREVIEWED AS OF JULY 10, 1980.
(8028 DEFICIENCY)

AS REQUIRED BY TECHNICAL SPECIFICATION 3.3.1, A MINIMUM OF 2 OPERABLE CHANNELS PER TRIP SYSTEM FOR THE SCRAM DISCHARGE VOLUME HIGH LEVEL TRIP IN MODES 1 AND 2 ARE TO BE OPERABLE. TECHNICAL SPECIFICATION 3.0.4 REQUIRES THAT ENTRY INTO AN OPERATIONAL CONDITION SHALL NOT BE MADE UNLESS LIMITING CONDITIONS FOR OPERATION OF THESE INSTRUMENTS ARE MET. CONTRARY TO THE ABOVE, THE REACTOR PLANT WAS TAKEN TO MODES 1 AND 2 WITH "A" AND "D" SCRAM DISCHARGE VOLUME LEVEL SWITCHES INOPERABLE SUBSEQUENT TO SCRAMS ON JUNE 2, JUNE 14, JULY 11, AND JULY 26, 1980.
(8035 INFRACTION)

INSPECTION STATUS

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

PLANT IS OPERATING AT FULL POWER.

+ LAST IE SITE INSPECTION DATE: SEPTEMBER 12, 1980

+ INSPECTION REPORT NO: 50-366/80-39

SUMMARY

INSPECTION SUMMARY

+ INSPECTION JUNE 16-27 (80-27): THIS SPECIAL, ANNOUNCED INSPECTION INVOLVED 182 INSPECTOR-HOURS ON SITE IN THE AREA OF HEALTH PHYSICS APPRAISAL INCLUDING RADIATION PROTECTION ORGANIZATION AND MANAGEMENT, PERSONNEL SELECTION, QUALIFICATIONS AND TRAINING, EXPOSURE CONTROLS, ALARA PROGRAM, RADIOACTIVE WASTE MANAGEMENT, FACILITIES AND EQUIPMENT, AND EMERGENCY RESPONSE CAPABILITIES. OF THE SEVEN AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN SIX AREAS; TWO ITEMS OF NONCOMPLIANCE WERE FOUND IN ONE AREA (INFRACTION - FAILURE TO FOLLOW WRITTEN PROCEDURES; INFRACTION - FAILURE TO PERFORM A SAFETY ANALYSIS PRIOR TO MAKING CHANGES TO THE FACILITY).

INSPECTION JULY 31 (80-32): INSPECTED CHEM-NUCLEAR WASTE DISPOSAL SITE, BARNWELL, SOUTH CAROLINA. EACH LICENSEE-SHIPPER VEHICLE WAS INSPECTED FOR COMPLIANCE WITH DEPARTMENT OF TRANSPORTATION (DOT) AND NUCLEAR REGULATORY COMMISSION (NRC) REGULATIONS AS FOLLOWS: (1) SHIPPING PAPER REQUIREMENTS; (2) DOT PLACARDING REQUIREMENTS; (3) RADIATION LEVELS; (4) REMOVABLE CONTAMINATION; (5) DOT MARKING AND LABELING REQUIREMENTS FOR PACKAGES; (6) DOT AND NRC REQUIREMENTS FOR EXTERNAL PACKAGE FEATURES, AND (7) PROHIBITED ARTICLES OR CONTENTS. OF THE SEVEN AREAS INSPECTED ONE ITEM OF NONCOMPLIANCE WAS OBSERVED IN ONE SHIPMENT (SEVERITY LEVEL IV - FAILURE TO PROPERLY PLACARD VEHICLE).

INSPECTION JULY 15 - AUGUST 15 (80-35): THIS INSPECTION INVOLVED 37 INSPECTOR-HOURS ON SITE IN THE AREAS OF TECHNICAL SPECIFICATION COMPLIANCE, REPORTABLE OCCURRENCES, HOUSEKEEPING, OPERATOR PERFORMANCE, OVERALL PLANT OPERATIONS. QUALITY ASSURANCE PRACTICES, STATION AND CORPORATE MANAGEMENT PRACTICES, CORRECTIVE AND PREVENTATIVE MAINTENANCE ACTIVITIES, SITE SECURITY PROCEDURES, RADIATION CONTROL ACTIVITIES AND SURVEILLANCE ACTIVITIES. OF THE ELEVEN AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN TEN AREAS. ONE ITEM OF NONCOMPLIANCE WAS FOUND IN ONE AREA (INFRACTION - REACTOR STARTUPS WITH INOPERATIVE SCRAM VOLUME LEVEL SWITCHES).

INSPECTION AUGUST 16 - SEPTEMBER 12 (80-39): THIS INSPECTION INVOLVED 44 INSPECTOR-HOURS ON SITE IN THE AREAS OF TECHNICAL SPECIFICATION COMPLIANCE, REPORTABLE OCCURRENCES, HOUSEKEEPING, OPERATOR PERFORMANCE, OVERALL PLANT OPERATIONS, QUALITY ASSURANCE PRACTICES, STATION AND CORPORATE MANAGEMENT PRACTICES, CORRECTIVE AND PREVENTATIVE MAINTENANCE ACTIVITIES AND SURVEILLANCE ACTIVITIES. OF THE ELEVEN AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

HUMBOLDT BAY

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	California/Humboldt 4 miles S of Eureka
2.	Electric Reliability Council	Western Systems Coordinating Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	220
5.	Design Electrical Rating (MWE Net)	65
6.	Date of Initial Criticality	February 16, 1963
7.	Date of Commercial Operation	August 1963
8.	Condenser Cooling Method	Circulating sea water
9.	Condenser Cooling Water Source	Pacific Ocean (Humboldt Bay)

Utility & Contractor Information

10.	Licensee	Pacific Gas & Electric Co.
11.	Corporate Address	77 Beale Street San Francisco, California 94106
12.	Corporate Contact	P. A. Crane, Jr. Asst. General Counsel
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	GE
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	V
17.	IE Branch Chief	J. L. Crews
18.	IE Section Chief	D. M. Sternberg
19.	IE Resident Inspector	J. D. Carlson
20.	Licensing Project Manager	V. L. Rooney
21.	Docket Number	50-133
22.	License Number/Date of Issue	DPR-7, 8/28/62
23.	Public Document Room	Humboldt County Library 636 F Street Eureka, California 95501

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

THE PLANT HAS BEEN SHUT DOWN SINCE JULY 1976.

ITEMS REMAINING TO BE COMPLETED DURING THE OUTAGE HAVE BEEN DEFERRED PENDING RESOLUTION OF THE SEISMIC ISSUES AT THE PLANT SITE. A DATE FOR RETURN TO OPERATIONS HAS NOT BEEN ESTABLISHED.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

PLANT IS SHUTDOWN.

LAST IE SITE INSPECTION DATE: 05/20-22/80

INSPECTION REPORT NO: 50-133/80-02

SUMMARY

INSPECTION SUMMARY

+ NO INSPECTIONS CONDUCTED IN SEPTEMBER 1980.

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-04/ 03L-0	08-27-80	09-06-80	FAILURE OF GE TYPE AK-1-15 CIRCUIT BREAKER.

LICENSE STATUS

Order issued 5/21/76 requiring Utility to prove seismic qualification before resuming operation of unit after outage of 7/2/76.

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 11-4-80

REVIEWED BY NRR DL: V. Rooney **DATE:** 11/6/80

HUMBOLDT BAY

AVERAGE DAILY POWER LEVEL (MWe)

OPERATING STATUS

1. Docket Number: 50-133
2. Utility Data Prepared By: B. C. Getty (707) 443-2787
3. Licensed Thermal Power (MWt): 220
4. Nameplate Rating (Gross MWe): 77 X 0.85 = 66
5. Design Electrical Rating (Net MWe): 65
6. Maximum Dependable Capacity (Gross MWe): 0
7. Maximum Dependable Capacity (Net MWe): 0
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons

None

9. Power Level To Which Restricted, If Any (Net MWe): None
10. Reasons For Restrictions, If Any:

This ... ...ve

11. Hours In Reporting Period
12. Number Of Hours Reactor Was Cri...
13. Reactor Reserve Shutdow...
14. Hours Generato...
15. Unit R...
16. ...H)
17. ...erated (MWH)
18. Net ...gy Generated (MWH)
19. Unit S... ...ce Factor
20. Unit Availability Factor
21. Unit Capacity Factor (Using MDC Net) 1/
22. Unit Capacity Factor (Using DER Net) 1/
23. Unit Forced Outage Rate
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

None

25. If Shut Down At End Of Report Period, Estimated Date of Startup: UNKNOWN

NO DECISION ON FUTURE OPERATION
SHUTDOWN INDEFINITELY

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
SEE SUMMARY BELOW									

SUMMARY

THE UNIT WAS SHUTDOWN ON 7/2/76 FOR SEISMIC MODIFICATIONS. THE UNIT REMAINS IN THE COLD SHUTDOWN CONDITION.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

INDIAN POINT 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	New York/Westchester 25 miles N of New York City
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2758
5.	Design Electrical Rating (MWE Net)	873
6.	Date of Initial Criticality	May 22, 1973
7.	Date of Commercial Operation	August 1973
8	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Hudson River

Utility & Contractor Information

10.	Licensee	Consolidated Edison Company
11.	Corporate Address	of New York, Inc. 4 Irving Place New York, New York
12.	Corporate Contact	P. Zarakas Vice President
13.	Architect/Engineer	UE&C
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	J. A. Jones/UE&C

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Senior Inspectc	H. B. KISTER
19.	IE Resident Inspector	T. Rebelowski (914-739-9360) FTS: 264-3311
20.	Licensing Project Manager	L. Olshan (X27144)
21.	Docket Number	50-247
22.	License Number/Date of Issue	DPR-26/9-28-73
23.	Public Document Room	White Plains Public Library 100 Martine Avenue White Plains, New York 10601

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS:

+ UNIT OPERATING ROUTINELY.

+ LAST IE SITE INSPECTION DATE: 9/24/80

+ INSPECTION REPORT NO: 50-247/80-16

SUMMARY

INSPECTION SUMMARY

+ 50-247/80-07 - MAY 11 - JUN 28: ROUTINE ONSITE, REGULAR AND BACKSHIFT INSPECTIONS BY THE RESIDENT INSPECTOR (53 HRS) OF PLANT OPERATIONS INCLUDING SHIFT LOGS AND RECORDS; LICENSEE ACTION ON PREVIOUS INSPECTION FINDINGS; OBSERVATION OF PHYSICAL SECURITY; REVIEW OF MONTHLY AND PERIODIC REPORTS; TRIPS; LICENSEE'S IMPLEMENTATION OF STRIKE PLANS; COMMISSIONER'S TOUR OF FACILITY; ACCIDENT MITIGATION FEATURES MEETING; REVIEW AND VERIFICATION OF INTERIM MEASURES - 120 DAY REPORTS; AND, RISK ASSESSMENT MEETING. NO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED.

+ 50-247/80-11 - JUL 30 - AUG 1: SPECIAL, ANNOUNCED INSPECTION (94 HRS) BY AN NRC:NRR/IE TEAM (5 PERSONS) OF UTILITY MANAGEMENT AND TECHNICAL COMPETENCE IN THE AREAS OF: SHIFT TECHNICAL ADVISORS; ONSITE AND OFFSITE ROUTINE AND EMERGENCY MANAGEMENT AND TECHNICAL RESOURCES CAPABILITY; TRAINING AND QUALIFICATIONS OF PERSONNEL; SAFETY REVIEW RESOURCES; TECHNICAL SUPPORT CENTER; ONSITE OPERATIONAL SUPPORT CENTER; DISSEMINATION OF OPERATING EXPERIENCES; AND OVERTIME POLICY. NO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED.

+ 50-247/80-12 - AUG 19-22: ROUTINE, UNANNOUNCED INSPECTION BY A REGION BASED INSPECTOR (35 HRS) OF QUALITY ASSURANCE PROGRAM IMPLEMENTATION INCLUDING: AUDITING; DESIGN CHANGES/MODIFICATIONS; AND PREVIOUSLY INSPECTOR IDENTIFIED ITEMS. NO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

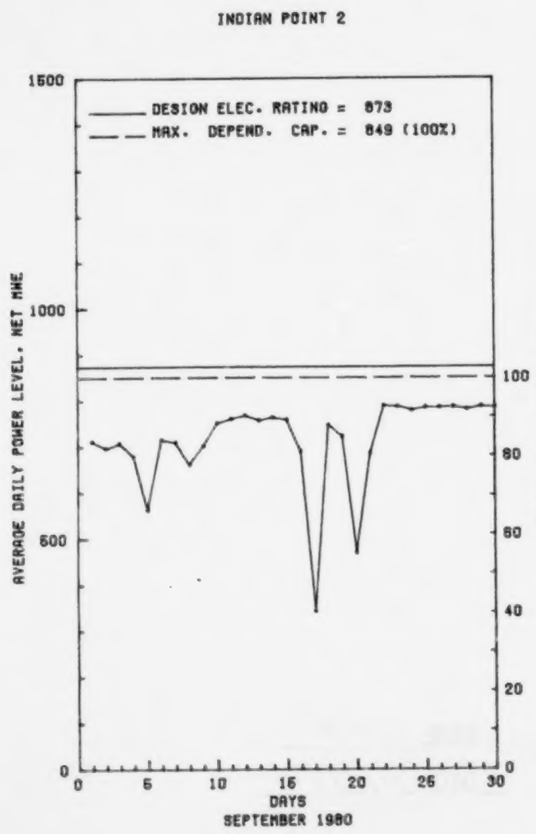
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-009/03L	08/10/80	09/10/80	DEFECTIVE OVER CURRENT TRIPPING DEVICE ASSOCIATED WITH WEST TYPE DB-75 BREAKER

REVIEWED BY OMPA LOEB: PE Bobe **DATE:** 11/4/80

REVIEWED BY NRR DL: Ross **DATE:** 11/6/80

INDIAN POINT 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-247
2. Utility Contact: E. EICH (914) 694-6000 X I.P.
3. Licensed Thermal Power (MWt): 2758
4. Nameplate Rating (Gross MWe): 1126 x 0.9 = 1013
5. Design Electrical Rating (Net MWe): 873
6. Maximum Dependable Capacity (Gross MWe): 885
7. Maximum Dependable Capacity (Net MWe): 849
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): 824
10. Reasons for Restrictions, If Any:
REMOVAL OF #3 DISC. (GEN. END) ON #23 TURBINE ROTOR.

ITEM 21 CALCULATED BY USING A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE*
11. Hours in Reporting Period	720.0	6,575.0	56,816.0
12. Hours Reactor Was Critical	720.0	5,474.9	37,765.4
13. Reactor Reserve Shutdown Hours	.0	883.5	1,124.9
14. Hours Generator On-Line	720.0	5,302.9	36,652.1
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,851,522	13,909,696	94,742,222
17. Gross Electrical Energy (MWH)	537,930	4,168,590	29,301,676
18. Net Electrical Energy (MWH)	513,940	3,974,335	27,939,435
19. Unit Service Factor	100.0	80.7	66.9
20. Unit Availability Factor	100.0	80.7	66.9
21. Unit Capacity Factor (MDC Net)	84.1	70.7	59.2
22. Unit Capacity Factor (DER Net)	81.8	69.2	58.4
23. Unit Forced Outage Rate	.0	9.4	8.3

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
CYCLE 4/5 REFUELING OUTAGE SCHEDULED 12/31/80 - 3 MOS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

*COMPUTED SINCE 7/1/74, THE DATE OF COMPLETION OF A 100 DAY – 100% POWER OPERATION TEST.

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	09/17/80	F	.0	A	5		HC	HTEXCH	BOTH CONDENSERS SERVING NO. 21 LP TURBINE OUT OF SERVICE DUE TO TUBE LEAKS IN 21 B CONDENSER & REPAIRS TO CWP ASSOCIATED WITH 21 A CONDENSER.
	09/20/80	S	.0	A	5		CH	PUMPXX	NO. 22 MAIN BOILER FEED PUMP SEAL WATER LINE REPAIR.

SUMMARY

INDIAN POINT 2 OPERATED NORMALLY IN SEPTEMBER, WITH TWO REDUCTIONS FOR CONDENSER TUBE LEAKS AND FEED PUMP SEAL WATER LINE REPAIR ON THE 17TH AND 20TH.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

INDIAN POINT 3

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	New York/Westchester 25 miles N of New York City
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	3025
5.	Design Electrical Rating (MWE Net)	965
6.	Date of Initial Criticality	April 6, 1976
7.	Date of Commercial Operation	August 30, 1976
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Hudson River

Utility & Contractor Information

10.	Licensee	Power Authority of the State of New York, Inc.
11.	Corporate Address	10 Columbus Circle New York, New York
12.	Corporate Contact	J. P. Bayne - Senior Vice President
13.	Architect/Engineer	UE&C
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	WEDCO

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Section Chief	H. B. KISTER
19.	IE Resident Inspector	T. Rebelowski (914-739-9360) FTS: 264-3311 - Operator
20.	Licensing Project Manager	L. Olshan x27144
21.	Docket Number	50-286
22.	License Number/Date of Issue	DPR-64, 12/12/75
23.	Public Document Room	White Plains Public Library 100 Martine Avenue White Plains, New York 10601

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

+ SYSTEM AND COMPONENT PROBLEMS:

+ THE LICENSEE EXPERIENCED TROUBLE WITH UNIT AUXILIARY TRANSFORMER AND IS CURRENTLY OPERATING USING THE STATION AUXILIARY TRANSFORMER.

+ FACILITY ITEMS (PLANS AND PROCEDURES):

+ A SHUTDOWN IS SCHEDULED ON OCTOBER 1 FOR A 45 DAY OUTAGE TO COMPLETE TMI RELATED MODIFICATIONS AND STEAM GENERATOR INSPECTIONS.

+ MANAGERIAL ITEMS:

+ THE SECURITY FORCE WAS CHANGED FROM AN OUTSIDE AGENCY TO THE CURRENT STATE SECURITY FORCE.

PLANT STATUS:

+ UNIT IS IN COASTDOWN FOR PLANT MODIFICATIONS.

+ LAST IE SITE INSPECTION DATE: 9/14-30/80

+ INSPECTION REPORT NO: 50-286/80-15

SUMMARY

INSPECTION SUMMARY

+ 50-286/80-12 - AUG 13-15: SPECIAL, ANNOUNCED INSPECTION (86 HRS) BY AN NRC NRR/IE TEAM (5 PERSONS) OF UTILITY MANAGEMENT AND TECHNICAL COMPETENCE IN THE AREAS OF SHIFT TECHNICAL ADVISORS; ONSITE AND OFFSITE ROUTINE AND EMERGENCY MANAGEMENT AND TECHNICAL RESOURCES CAPABILITY; TRAINING AND QUALIFICATIONS OF PERSONNEL; SAFETY REVIEW RESOURCES; DISSEMINATION OF OPERATING EXPERIENCES; AND OVERTIME POLICY. NO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

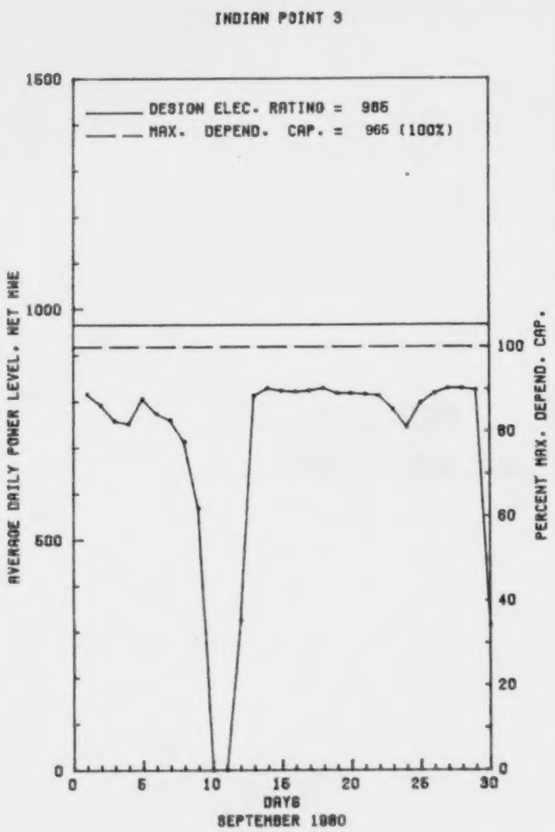
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-001/ EXP	06/24/80	07/11/80	EXPOSURE OF THREE INDIVIDUALS WHILE REPAIRING INCORE FLUX MONITORING EQUIPMENT

REVIEWED BY OMPA LOEB: PE Kobe DATE: 11/4/80

REVIEWED BY NRR DL: W J Ross DATE: 11/6/80

INDIAN POINT 3

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-286
2. Utility Contact: C. Connell (914) 739-8200 x242
3. Licensed Thermal Power (MWt): 3025
4. Nameplate Rating (Gross MWe): 1126 x 0.9 = 1013
5. Design Electrical Rating (Net MWe): 965
6. Maximum Dependable Capacity (Gross MWe): 1000
7. Maximum Dependable Capacity (Net MWe): 965
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): 917
10. Reasons for Restrictions, If Any:
REMOVED SOME TURBINE BLADING

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	6,575.0	35,832.0
12. Hours Reactor Was Critical	668.2	4,855.8	26,448.7
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	658.8	4,386.7	25,419.5
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,905,318	10,800,817	66,905,284
17. Gross Electrical Energy (MWH)	538,310	3,017,310	21,382,611
18. Net Electrical Energy (MWH)	515,900	2,871,981	20,514,518
19. Unit Service Factor	91.5	66.7	70.9
20. Unit Availability Factor	91.5	66.7	70.9
21. Unit Capacity Factor (MDC Net)	74.3	45.3	59.3
22. Unit Capacity Factor (DER Net)	74.3	45.3	59.3
23. Unit Forced Outage Rate	8.5	28.1	7.4

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
TURBINE OUTAGE OCTOBER 1, 1980, 45 DAYS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
28	09/09/80	F	61.2	A	3		ED	INSTRU	UNIT TRIP #32 STEAM GENERATOR MISMATCH CAUSED BY LOSS OF 33 STATIC INVERTER, REPLACED CAPACITORS ON STATIC INVERTER. OUTAGE EXTENDED DUE TO FAULT ON UNIT AUXILIARY TRANSFORMER. UNIT AUXILIARY TRANSFORMER PROBLEM BEING ANALYZED TO DETERMINE FAULT.

SUMMARY

INDIAN POINT III OPERATED WITH ONE OUTAGE ON THE 9TH AND A COASTDOWN ON THE 30TH FOR A TURBINE OUTAGE.

LEGEND

[1]Type:
F--Forced
S--Scheduled

[2]Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3]Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4]Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5]Exhibit H:
Same source.

KEWAUNEE

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Wisconsin/Kewaunee 27 miles E of Green Bay
2.	Electric Reliability Council	Mid-America Interpool Network
3.	Type of Reactor	PWR
4.	Power Level (MWT)	1650
5.	Design Electrical Rating (MWE Net)	535
6.	Date of Initial Criticality	March 7, 1974
7.	Date of Commercial Operation	June 1974
8.	Condenser Cooling Method	Once through condenser and return to Lake Michigan
9.	Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10.	Licensee	Wisconsin Public Service Corp.
11.	Corporate Address	P. O. Box 1200 Green Bay, Wisconsin 54305
12.	Corporate Contact	E. R. Mathews Power Supply & Eng.
13.	Architect/Engineer	Fluor Power Services Corp.
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Pioneer Services & Eng. Co.

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	R. F. Warnick
19.	IE Resident Inspector	R. L. Nelson
20.	Licensing Project Manager	R. B. Licciardo X28362
21.	Docket Number	50-305
22.	License Number/Date of Issue	DPR-43 issued 12/21/73
23.	Public Document Room	Kewaunee Public Library 822 Juneau Street Kewaunee, Wisconsin 54216

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

+ The plant operated at power during most of the month. At months end the unit was shutdown to repair valves in the reactor coolant system RTD loop.

LAST IE SITE INSPECTION DATE: July 1-31, 1980

INSPECTION REPORT NO: 80-16

SUMMARY

INSPECTION SUMMARY

+ No inspections conducted.

REPORTS RECEIVED FROM LICENSEE

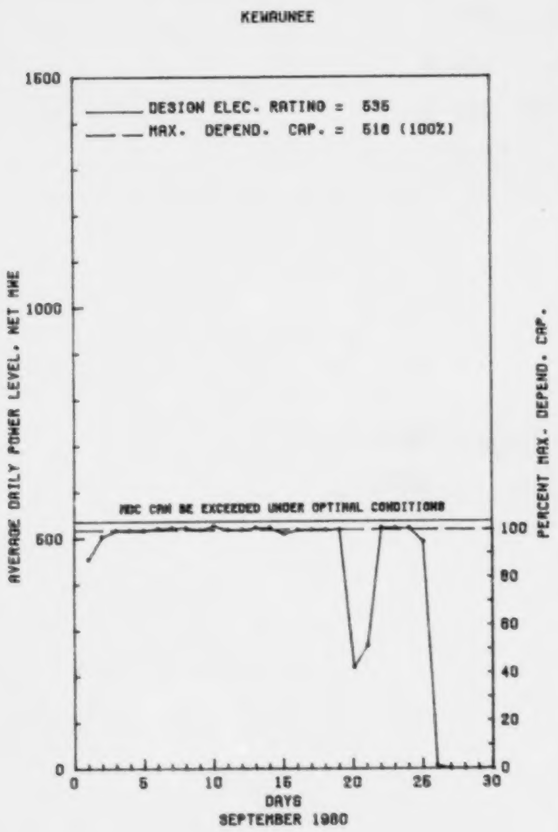
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE			

REVIEWED BY OMPA LOEB: P C Bole DATE: 11/4/80

REVIEWED BY NRR DL: R. Licciardo DATE: 11/6/80

KEWAUNEE

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-305
2. Utility Contact: G.RUITER (414) 388-2560 X207
3. Licensed Thermal Power (MWt): 1650
4. Nameplate Rating (Gross MWe): 622 x 0.9 = 560
5. Design Electrical Rating (Net MWe): 535
6. Maximum Dependable Capacity (Gross MWe): 545
7. Maximum Dependable Capacity (Net MWe): 518
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

ITEM 21 CALCULATED BY USING A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	6,575.0	55,176.0
12. Hours Reactor Was Critical	602.2	5,122.9	46,125.4
13. Reactor Reserve Shutdown Hours	.0	152.7	2,330.5
14. Hours Generator On-Line	601.5	5,029.5	45,025.5
15. Unit Reserve Shutdown Hours	.0	.0	10.0
16. Gross Thermal Energy (MWH)	946,334	7,982,581	69,133,704
17. Gross Electrical Energy (MWH)	310,200	2,639,000	22,822,800
18. Net Electrical Energy (MWH)	295,198	2,514,923	21,714,001
19. Unit Service Factor	83.6	76.5	81.6
20. Unit Availability Factor	83.6	76.5	81.6
21. Unit Capacity Factor (MDC Net)	79.2	73.1	75.1
22. Unit Capacity Factor (DER Net)	76.6	71.5	73.6
23. Unit Forced Outage Rate	16.4	8.2	5.6

24. Shutdowns Scheduled Over Next 6 Months (Type,Date,and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 10/01/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type^1	Duration (Hours)	Reason^2	Method of Shutting Down Reactor^3	Licensee Event Report No.	System Code^4	Component Code^5	Cause & Corrective Action to Prevent Recurrence
8	09/20/80	S	.0	B	5		HF	HTEXCH	A LOAD REDUCTION TO 45% POWER WAS TAKEN OVER THE WEEKEND TO PERMIT INSPECTION OF THE MAIN CONDENSER WATER BOXES.
9	09/26/80	F	118.1	A	1	80-032	CB	VALVEX	A DISC/STEM SEPARATION IN A LOOP B RTD BY-PASS LOOP ISOLATION VALVE CAUSED LOSS OF FLOW IN THE RTD LINE. THE UNIT WAS TAKEN TO COLD SHUTDOWN TO PERMIT REPLACEMENT OF THE FAILED VALVE. THE OUTAGE WAS EXTENDED WHEN ANOTHER LOOP B RTD BY-PASS LOOP ISOLATION VALVE FAILED IN THE SAME MANNER AFTER IT WAS OPERATED TO ISOLATE THE BY-PASS LOOP DURING MAINTENANCE. THIS VALVE WAS ALSO REPLACED.

SUMMARY

KEWAUNEE OPERATED AT OR NEAR FULL POWER FOR MOST OF THE MONTH WITH ONE REDUCTION FOR CONDENSER WATER BOX INSPECTION ON THE 20TH AND AN OUTAGE ON THE 27TH FOR VALVE REPLACEMENT THROUGH THE END OF SEPTEMBER.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram.
3--Automatic Scram.
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

LACROSSE

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Wisconsin/Vernon 19 miles S of LaCrosse
2.	Electric Reliability Council	Mid-Continent Area Reliability Coordination Agreement
3.	Type of Reactor	BWR
4.	Power Level (MWT)	165
5.	Design Electrical Rating (MWE Net)	50
6.	Date of Initial Criticality	July 11, 1967
7.	Date of Commercial Operation	November 1, 1969
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Mississippi River

Utility & Contractor Information

10.	Licensee	Dairyland Power Cooperative
11.	Corporate Address	2615 East Avenue South La Crosse, Wisconsin
12.	Corporate Contact	F. Linder
13.	Architect/Engineer	Sargent & Lundy
14.	Nuclear Steam Supply System	Allis-Chalmers
15.	Constructor	Maxson Construction

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. F. HEISHMAN
18.	IE Section Chief	D. C. BOYD
19.	IE Resident Inspector	W. Forney
20.	Licensing Project Manager	J. J. Shea
21.	Docket Number	50-409
22.	License Number/Date of Issue	DPR-45 issued 8/28/73
23.	Public Document Room	LaCrosse Public Library 800 Main Street LaCrosse, Wisconsin 54601

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

MANAGERIAL ITEMS

+ Mr. Paul Sheffer was employed as Radiation Protection Engineer about June 1, 1980. (80-08)

PLANT STATUS

Routine operation.

+LAST IE SITE INSPECTION DATE: August 26-28, 1980

+ INSPECTION REPORT NO: 80-08

INSPECTION SUMMARY

+ Inspection on August 26-28, (80-08): Routine, unannounced inspection of Environmental Protection Program including: management controls; quality control of analytical measurements; implementation of Environmental Monitoring Program; and Confirmatory Measurements Program including: a discussion of results of comparative analyses of previous radiological effluent samples; collection of effluent samples for subsequent comparative analysis. The inspection involved 20 inspector-hours onsite by one NRC inspector. No apparent items of noncompliance or deviations were identified.

REPORTS RECEIVED FROM LICENSEE

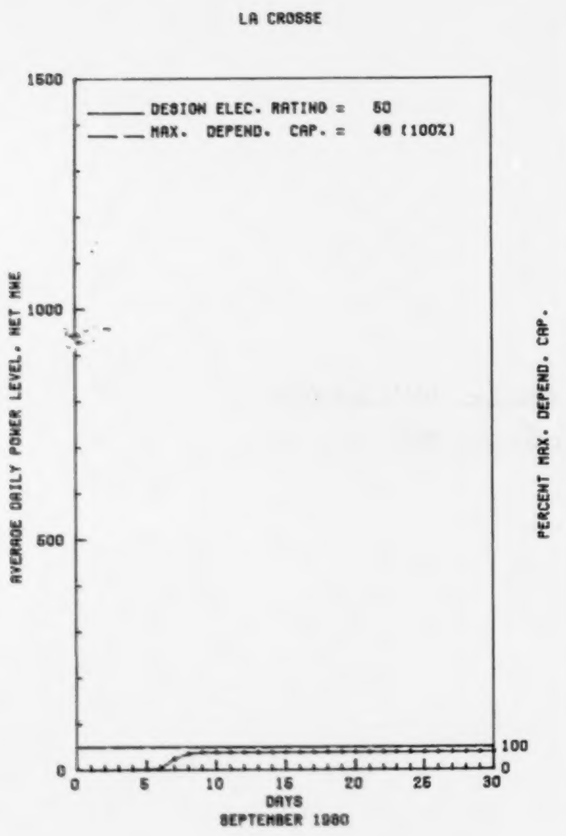
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-05/ 01T-0	08/06/80	08/18/80	During operation, valve addition to shutdown condenser shell gauge tap found without design change, no checklist identified valve.
80-06/ 03L-0	08/14/80	09/12/80	During reactor shutdown, indicated gross alpha activity of primary coolant sample exceeded TS limits.
80-07/ 03L-0	08/16/80	09/12/80	During reactor shutdown, indicated gross alpha activity of primary coolant sample exceeded TS limits.
80-08/ 03L-0	08/19/80	09/12/80	During normal operation, the 1B high pressure service water diesel did not staft upon manual activation of control room control switch.

REVIEWED BY OMPA LOEB: John J. Rizzo DATE: 11-4-80

REVIEWED BY NRR DL: J J Shea DATE: 11-6-80

LACROSSE

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-409
2. Utility Contact: L. S. Goodman (608) 689-2331
3. Licensed Thermal Power (MWt): 165
4. Nameplate Rating (Gross MWe): 76.8 x 0.85 =65
5. Design Electrical Rating (Net MWe): 50
6. Maximum Dependable Capacity (Gross MWe): 50
7. Maximum Dependable Capacity (Net MWe): 48
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	6,575.0	95,690.0
12. Hours Reactor Was Critical	644.7	5,347.6	63,153.7
13. Reactor Reserve Shutdown Hours	.0	.0	478.0
14. Hours Generator On-Line	593.6	5,095.4	58,140.4
15. Unit Reserve Shutdown Hours	.0	.0	79.0
16. Gross Thermal Energy (MWH)	80,701	677,968	7,962,747
17. Gross Electrical Energy (MWH)	23,314	196,185	2,395,317
18. Net Electrical Energy (MWH)	21,638	182,435	2,215,170
19. Unit Service Factor	82.4	77.5	60.7
20. Unit Availability Factor	82.4	77.5	60.8
21. Unit Capacity Factor (MDC Net)	62.6	57.8	48.2
22. Unit Capacity Factor (DER Net)	60.1	55.5	46.3
23. Unit Forced Outage Rate	17.6	14.6	7.2

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): ESTIMATED REFUELING OUTAGE, NOVEMBER 16, 1980, 6 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-08	08/22/80	F	94.3	A	4		CH	INSTRU	THE AUGUST OUTAGE DUE TO THE TRIPPING OF 1A REACTOR FEED PUMP WAS COMPLETED. WHILE THE PLANT WAS SHUTDOWN, MAINTENANCE WAS ALSO PERFORMED ON THE SEAL INJECT SYSTEM AND THE MECHANICAL SEALS IN CONTROL ROD DRIVE MECHANISMS NOS. 5 AND 21.
80-09	09/05/80	F	32.1	A	3		RB	CRDRVE	WATER LEAKAGE FROM THE SEAL INJECT SUPPLY LINE CONNECTION TO CONTROL ROD DRIVE MECHANISM NO. 1 SPRAYED ONTO THE SCRAM SOLENOID AREA OF CRDM NO. 3, CAUSING AN ELECTRICAL SHORT, RESULTING IN A PARTIAL SCRAM. THE SEAL INJECT LINE CONNECTION AT CRDM NO. 1 WAS TIGHTENED AND INBOARD SCRAM SOLENOID COIL ON CRDM NO. 3 WAS REPLACED.

SUMMARY

LACROSSE RETURNED ON-LINE ON THE 5TH FROM A MAINTENANCE/REPAIR OUTAGE AND THEN INCURRED ANOTHER OUTAGE ON THE 5TH FOR REPAIR OF ELECTRICAL SHORT OF CONTROL ROD DRIVE MECHANISM AND WATER LEAK RESPONSIBLE FOR SHORT CIRCUIT. AFTER REPAIRS, LACROSSE OPERATED NORMALLY FOR REMAINDER OF THE MONTH.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

MAINE YANKEE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Maine/Lincoln 10 miles N of Bath (near Wiscasset)
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2630
5.	Design Electrical Rating (MWE Net)	825
6.	Date of Initial Criticality	October 23, 1972
7.	Date of Commercial Operation	December 28, 1972
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Back River (Atlantic)

Utility & Contractor Information

10.	Licensee	Maine Yankee Atomic Power Co.
11.	Corporate Address	20 Turnpike Road Westboro, Massachusetts 01581
12.	Corporate Contact	D. W. Edwards, Assistant to Vice President
13.	Architect/Engineer	Stone & Webster
14.	Nuclear Steam Supply System	Combustion Engineering
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	IE Section Chief	T. T. Martin
19.	IE Resident Inspector	W. J. LAZARUS (207) 882-7519
20.	Licensing Project Manager	G. Requa (X28478)
21.	Docket Number	50-309
22.	License Number/Date of Issue	DPR-36, 9/16/72
23.	Public Document Room	Wiscasset Public Library High Street Wiscasset, Maine 04578

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

- SYSTEM AND COMPONENT PROBLEMS:
- FAILURE OF THE POWER SUPPLY FOR #41 CEDM UPPER GRIPPER COIL RESULTED IN A PLANT TRIP ON SEPTEMBER 6, 1980. BOTH CEDM MOTOR GENERATOR CIRCUIT BREAKERS OPENED WHEN THE POWER SUPPLY SHORTED TO GROUND.
- THE SECOND OF THREE MECHANICAL SEALS FAILED ON #3 RCP ON SEPTEMBER 14, 1980. A CONTROLLED SHUTDOWN AND COOLDOWN WAS PERFORMED TO REPLACE THE SEAL.

PLANT STATUS:

- THE PLANT RETURNED TO POWER ON SEPTEMBER 19, 1980, FOLLOWING A 5 DAY OUTAGE FOR REPLACEMENT OF #3 RCP SEAL, AND CONTINUES TO OPERATE AT NEAR FULL POWER.
- LAST IE SITE INSPECTION DATE: 9/2-30/80
- INSPECTION REPORT NO: 50-309/80-14

SUMMARY

INSPECTION SUMMARY

- 50-309/80-13 (15000039/80-07) - MAY 16: VISIT TO CHEM-NUCLEAR WASTE DISPOSAL SITE, BARNWELL, SOUTH CAROLINA BY ONE REGION BASED INSPECTOR FOR UNANNOUNCED INSPECTIONS OF LICENSEE-SHIPPERS. EACH LICENSEE-SHIPPER VEHICLE WAS INSPECTED FOR COMPLIANCE WITH DEPARTMENT OF TRANSPORTATION (DOT) AND NUCLEAR REGULATORY COMMISSION (NRC) REGULATIONS AS FOLLOWS: (1) SHIPPING PAPER REQUIREMENTS; (2) DOT PLACARDING REQUIREMENTS; (3) RADIATION LEVELS; (4) REMOVABLE CONTAMINATION; (5) DOT MARKING AND LABELING REQUIREMENTS FOR PACKAGES; (6) DOT AND NRC REQUIREMENTS FOR EXTERNAL PACKAGE FEATURES, AND (7) PROHIBITED ARTICLES OR CONTENTS. ONE ITEM OF NONCOMPLIANCE WAS IDENTIFIED: FAILURE TO MARK PACKAGES "RADIOACTIVE-LSA" TO IDENTIFY CONTENTS.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

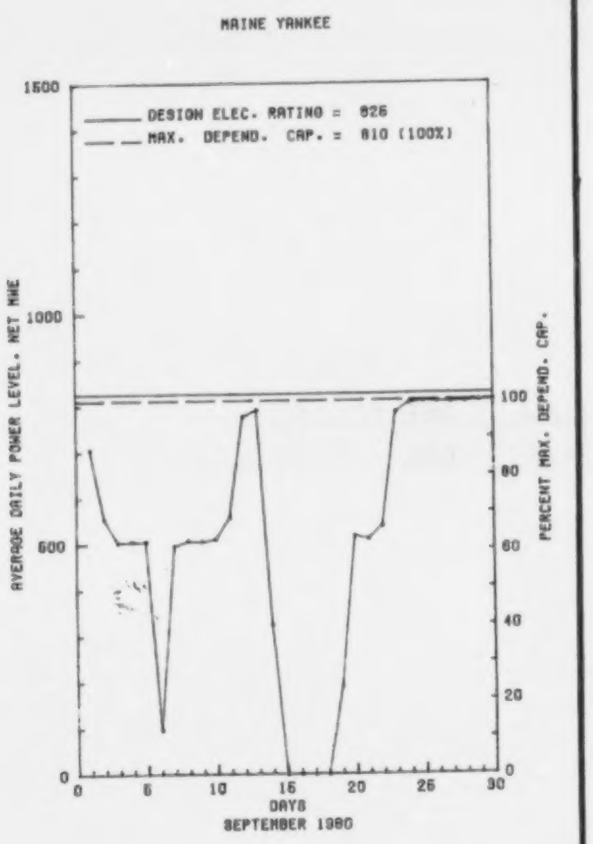
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE			

REVIEWED BY OMPA LOEB: DATE: 11/04/80

REVIEWED BY NRR DL: C Nelson DATE: 11-6-80

MAINE YANKEE

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-309
2. Utility Contact: F.C. BEERS (617) 366-9011
3. Licensed Thermal Power (MWt): 2630
4. Nameplate Rating (Gross MWe): 900 x 0.9 = 810
5. Design Electrical Rating (Net MWe): 825
6. Maximum Dependable Capacity (Gross MWe): 850
7. Maximum Dependable Capacity (Net MWe): 810
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

CAPACITY FACTORS CALCULATED WITH A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	6,575.0	69,203.6
12. Hours Reactor Was Critical	595.0	4,668.3	55,873.2
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	590.0	4,469.1	53,986.8
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,219,584	9,516,074	114,756,157
17. Gross Electrical Energy (MWH)	392,850	3,106,510	37,679,690
18. Net Electrical Energy (MWH)	372,535	2,951,810	35,778,996
19. Unit Service Factor	81.9	68.0	78.0
20. Unit Availability Factor	81.9	68.0	78.0
21. Unit Capacity Factor (MDC Net)	63.9	55.4	86.7
22. Unit Capacity Factor (DER Net)	62.7	54.4	84.7
23. Unit Forced Outage Rate	18.1	10.4	7.2

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	09/01/80	F	8.0	A	5		HH	MECFUN	LOAD REDUCTION TO 60% TO REPAIR LEAKING COUPLING ON THE P-2 FEEDWATER PUMP.
	09/02/80	F	236.5	A	5		HH	XXXXXX	LOAD REDUCTION TO 65% TO CHANGE LUBE OIL IN P-2A FEEDWATER PUMP AND INVESTIGATE HIGHER-THAN-NORMAL TEMPERATURE ON ONE OF THE SPEED-CHANGER'S BEARINGS. BEARING WAS FOUND TO BE WIPED AND WAS REPAIRED.
12-80	09/06/80	F	14.5	A	3		IA	CKTBRK	PLANT TRIP DUE TO SPURIOUS OPENING OF THE ROD DRIVE MG-SET OUTPUT BREAKERS. INVESTIGATION FAILED TO DETERMINE EXACT CAUSE.
13-80	09/14/80	F	115.5	A	1		CB	PUMPXX	FAILURE OF 2 OF THE 4 RCP SEAL STAGES REQUIRED SHUTDOWN AND REPLACEMENT OF THE SEAL CARTRIDGE UNIT.
	09/19/80	F	81.3	A	5		HH	XXXXXX	HOLDING LOAD @ APPROX 64% DURING POWER ESCALATION FROM S/D #13-80-5 DUE TO BEARING TEMPERATURE PROBLEM AND OIL LEAKS ON P-2A FEEDWATER PUMP. OIL SYSTEM FLUSHED AND LEAKS REPAIRED.

SUMMARY

MAINE YANKEE OPERATED WITH THREE LOAD REDUCTIONS AND TWO OUTAGES FOR REPAIR WORK DURING THE MONTH OF SEPTEMBER.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram.
3--Automatic Scram.
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

MILLSTONE 1

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Connecticut/New London 5 miles SW of New London
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2011
5.	Design Electrical Rating (MWE Net)	660
6.	Date of Initial Criticality	October 26, 1970
7.	Date of Commercial Operation	March 1971
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Long Island Sound

Utility & Contractor Information

10.	Licensee	Northeast Nuclear Energy Company
11.	Corporate Address	P. O. Box 270 Hartford, Connecticut
12.	Corporate Contact	D. Switzer, President
13.	Architect/Engineer	Ebasco
14.	Nuclear Steam Supply System	GE
15.	Constructor	Ebasco

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	IE Section Chief	R. KEIMIG
19.	IE Resident Inspector	J. SHEDLOSKY (203-447-3179) FTS: 642-7287
20.	Licensing Project Manager	J. J. Shea
21.	Docket Number	50-245
22.	License Number/Date of Issue	DPR-21, 10/7/70
23.	Public Document Room	Waterford Public Library Rope Ferry Road Route 156 Waterford, Connecticut 06385

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

+ FACILITY ITEMS (PLANS AND PROCEDURES):

+ A REFUELING OUTAGE IS SCHEDULED TO COMMENCE ON 10/4 AND IS EXPECTED TO LAST APPROXIMATELY TEN WEEKS.

+ MANAGERIAL ITEMS:

+ MEMBERS OF LICENSEE MANAGEMENT MET WITH NRC REGION I STAFF ON SEPTEMBER 11, 1980 TO DISCUSS LICENSEE PERFORMANCE. THIS MEETING WAS CONDUCTED AS PART OF THE SYSTEMATIC ASSESSMENT OF LICENSEE PERFORMANCE (SALP) PROGRAM.

PLANT STATUS:

+ ROUTINE FULL POWER OPERATION WITH AN OFFGAS RATE OF 150,000 MICROCURIES PER SECOND AND A STACK RELEASE RATE OF 150 MICROCURIES PER SECOND.

+ LAST IE SITE INSPECTION DATE: 9/22-26/80

+ INSPECTION REPORT NO: 50-245/80-16

SUMMARY

INSPECTION SUMMARY

+ 50-245/80-11 - JUL 6 - AUG 2: ROUTINE, ONSITE, REGULAR AND BACKSHIFT INSPECTION BY TWO RESIDENT INSPECTORS (75 HRS). AREAS INSPECTED INCLUDED THE CONTROL ROOMS AND THE ACCESSIBLE PORTIONS OF THE REACTOR, TURBINE, RADIOACTIVE WASTE, GAS TURBINE GENERATOR, AND INTAKE BUILDINGS; RADIATION PROTECTION; PHYSICAL SECURITY; FIRE PROTECTION; PLANT OPERATING RECORDS; SURVEILLANCE TESTING; CALIBRATION; MAINTENANCE; CORE POWER DISTRIBUTION LIMITS AND REPORTING TO THE NRC. TWO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED: (1) FAILURE TO REQUIRE A RADIATION WORK PERMIT IN ACCORDANCE WITH H.P. PROCEDURES; (2) FAILURE TO INSTRUCT WORKERS OF RADIOACTIVE MATERIALS IN THE WORK AREA.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

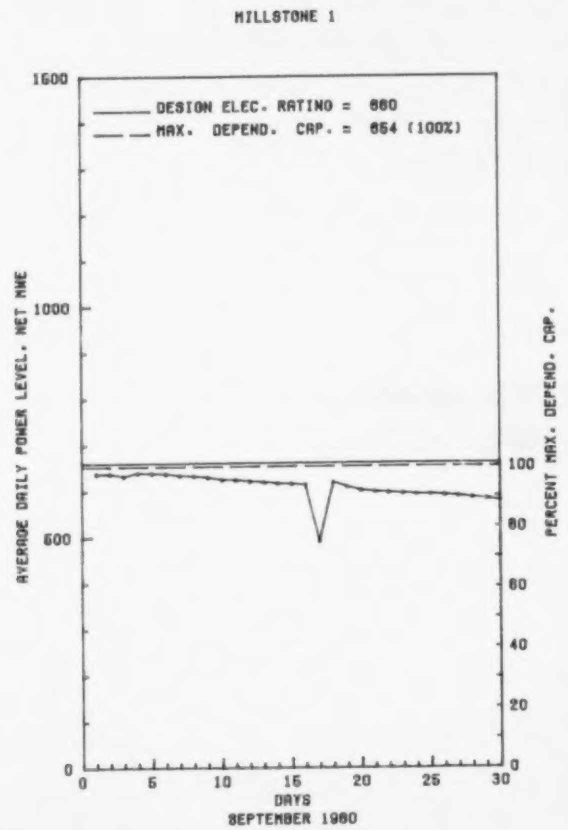
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-012/03L	07/25/80	08/22/80	ONE OF SIX APRM CHANNELS READING 10 PERCENT LOW

REVIEWED BY OMPA LOEB: John J. Riggs **DATE:** 11-4-80

REVIEWED BY NRR DL: J. J. Shea **DATE:** 11-6-80

MILLSTONE 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-245
2. Utility Contact: G. HARRAN (203) 447-1792 X655
3. Licensed Thermal Power (MWt): 2011
4. Nameplate Rating (Gross MWe): 735 x 0.9 = 662
5. Design Electrical Rating (Net MWe): 660
6. Maximum Dependable Capacity (Gross MWe): 684
7. Maximum Dependable Capacity (Net MWe): 654
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
END OF CYCLE COASTDOWN

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	6,575.0	86,255.0
12. Hours Reactor Was Critical	720.0	5,986.0	66,371.0
13. Reactor Reserve Shutdown Hours	.0	592.3	1,527.3
14. Hours Generator On-Line	720.0	5,954.0	64,049.9
15. Unit Reserve Shutdown Hours	.0	26.5	26.5
16. Gross Thermal Energy (MWH)	1,369,333	10,359,790	114,407,735
17. Gross Electrical Energy (MWH)	459,500	3,519,447	38,790,843
18. Net Electrical Energy (MWH)	438,444	3,360,650	37,000,432
19. Unit Service Factor	100.0	90.6	74.3
20. Unit Availability Factor	100.0	91.0	74.3
21. Unit Capacity Factor (MDC Net)	93.1	78.2	65.6
22. Unit Capacity Factor (DER Net)	92.3	77.4	65.0
23. Unit Forced Outage Rate	.0	.2	15.3

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
REFUEL & MAINTENANCE - 10/04/80 - 11 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
MILLSTONE 1									
09	09/17/80	S	.0	B	5		HC	HTEXCH	REDUCED POWER TO REPAIR MAIN CONDENSER TUBE LEAKS. POWER WAS INCREASED FOLLOWING MAINTENANCE.

SUMMARY

MILLSTONE 1 OPERATED NORMALLY FOR THE REPORTING PERIOD, WITH ONE REDUCTION ON THE 17TH FOR MAIN CONDENSER TUBE LEAK REPAIR.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

MILLSTONE 2

FACILITY DATA

Facility Description		
1.	Location (state/county) (nearest population center)	Connecticut/New London 5 miles SW of New London
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2700
5.	Design Electrical Rating (MWE Net)	870
6.	Date of Initial Criticality	October 17, 1975
7.	Date of Commercial Operation	December 26, 1975
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Long Island Sound
Utility & Contractor Information		
10.	Licensee	Northeast Nuclear Energy Company
11.	Corporate Address	P. O. Box 270 Hartford, Connecticut
12.	Corporate Contact	D. Switzer, President
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Combustion Engineering
15.	Constructor	Bechtel
Regulatory Information		
16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	IE Section Chief	R. Keimig
19.	IE Resident Inspector	J. T. SHEDLOSKY (203-447-3179) FTS: 642-7287 R. Zimmerman
20.	Licensing Project Manager	E. L. Conner, Jr.
21.	Docket Number	50-336
22.	License Number/Date of Issue	DPR-65, 8/1/75
23.	Public Document Room	Waterford Public Library Rope Ferry Road Route 156 Waterford, Connecticut 06385

INSPECTION STATUS

ENFORCEMENT STATUS

UNIT 2 TECHNICAL SPECIFICATION 3.7.8.1 STATES, "ALL HYDRAULIC SNUBBERS LISTED IN TABLE 3.7-1 SHALL BE OPERABLE." SURVEILLANCE REQUIREMENT 4.7.8.1 STATES, IN PART, "(A) HYDRAULIC SNUBBERS SHALL BE DEMONSTRATED OPERABLE BY PERFORMANCE OF THE FOLLOWING AUGMENTED INSERVICE INSPECTION PROGRAM. (B) EACH HYDRAULIC SNUBBER WITH SEAL MATERIAL NOT FABRICATED FROM ETHYLENE PROPYLENE OR OTHER MATERIAL DEMONSTRATED COMPATIBLE WITH THE OPERATING ENVIRONMENT SHALL BE DETERMINED OPERABLE AT LEAST ONCE PER 31 DAYS BY A VISUAL INSPECTION OF THE SNUBBER." CONTRARY TO THE ABOVE, THE REACTOR WAS OPERATED IN MODES 1 THROUGH 4 DURING A PERIOD FROM DECEMBER 4 THROUGH 22, 1979, WITH TWO HYDRAULIC SNUBBERS (HANGER NUMBERS 413082 AND 490031) LISTED IN TABLE 3.7-1 WHICH HAVE SEALS FABRICATED FROM MATERIAL OTHER THAN ETHYLENE PROPYLENE, WHICH WERE NOT DEMONSTRATED TO BE COMPATIBLE WITH THE ENVIRONMENT AND HAD NOT BEEN INSPECTED WITHIN THE PREVIOUS 31 DAYS.
(8004 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

+ MANAGERIAL ITEMS:

+ MEMBERS OF LICENSEE MANAGEMENT MET WITH NRC REGION I STAFF ON SEPTEMBER 11, 1980 TO DISCUSS LICENSEE PERFORMANCE. THIS MEETING WAS CONDUCTED AS PART OF THE SYSTEMATIC ASSESSMENT OF LICENSEE PERFORMANCE (SALP) PROGRAM.

PLANT STATUS:

+ THE REFUELING OUTAGE COMMENCED 8/16. CRITICALITY AND FULL POWER OPERATION ARE TENTATIVELY SCHEDULED FOR 10/6 AND 10/20, RESPECTIVELY.

+ LAST IE SITE INSPECTION DATE: 9/22-26/80

+ INSPECTION REPORT NO: 50-336/80-17

SUMMARY

INSPECTION SUMMARY

+ 50-336/80-09 - JUL 6 - AUG 2: ROUTINE, ONSITE, REGULAR AND BACKSHIFT INSPECTION BY TWO RESIDENT INSPECTORS (67 HRS). AREAS INSPECTED INCLUDED THE CONTROL ROOM, THE ENCLOSURE, AUXILIARY, TURBINE AND INTAKE BUILDINGS; AND THE CONDENSATE POLISHING FACILITY; RADIATION PROTECTION; PHYSICAL SECURITY; FIRE PROTECTION; PLANT OPERATING RECORDS; SURVEILLANCE TESTING; CALIBRATION; MAINTENANCE; CORE POWER DISTRIBUTION LIMITS; AND REPORTING TO THE NRC. ONE ITEM OF NONCOMPLIANCE WAS IDENTIFIED: FAILURE TO REQUIRE RESTORATION OF EQUIPMENT FOLLOWING MAINTENANCE.

+ 50-336/80-13 - AUG 27-29: ROUTINE, UNANNOUNCED INSPECTION BY A REGION-BASED INSPECTOR (14 HRS) OF INSERVICE INSPECTION ACTIVITIES. NO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

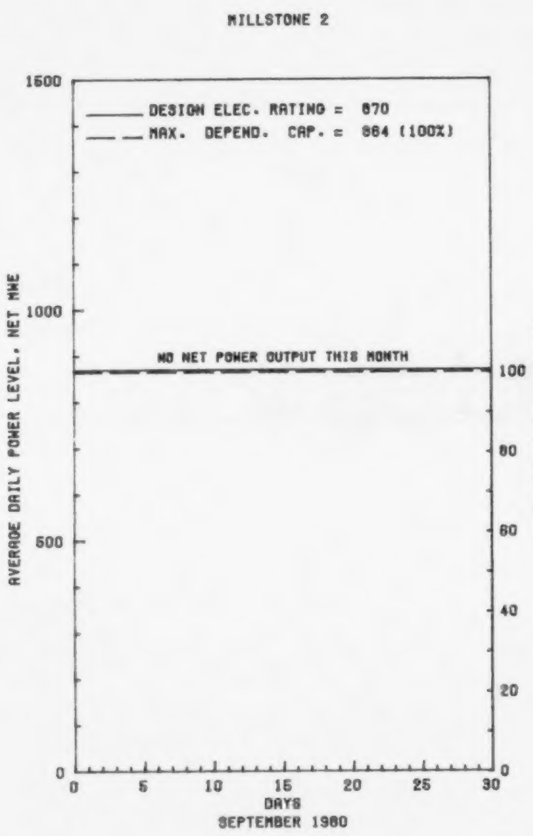
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-026/ 01T	07/29/80	08/12/80	DISCOVERY OF CONDITIONS NOT ADDRESSED IN SAR OR TECHNICAL SPECIFICATIONS - BUILDING FLOODING DUE TO RAIN WATER INLEAKAGE THROUGH PIPING PENETRATIONS
80-027/ 03L	07/29/80	08/25/80	FAILED SOCKET WELD ON A FITTING IN A ONE INCH SAFETY INJECTION TANK RECIRCULATION LINE
80-028/ 03L	08/11/80	09/08/80	DROP OF CEA NOS. 41 AND 65 DUE TO FAULTY JUMPER CABLE FROM REFUEL DISCONNECT PANEL TO THE REACTOR HEAD. RESULTED IN FLUX TILT GREATER THAN 0.02
80-029/ 03L	08/12/80	09/08/80	EMERGENCY DIESEL GENERATOR 12V OUT OF SERVICE DUE TO GROUND IN CONTROL WIRING
80-030/ 03L	08/12/80	09/08/80	CONTAINMENT AIR TEMPERATURE FOUND TO BE IN EXCESS OF TECHNICAL SPECIFICATIONS 120 DEGREES F. TEMPERATURE CONTROL VALVE ON CLOSED COOLING SYSTEM SET TOO HIGH

REVIEWED BY OMPA LOEB: MR Beebe DATE: 11/04/80

REVIEWED BY NRR DL: EL Conner DATE: 11/5/80

MILLSTONE 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-336
2. Utility Contact: G. H. Howlett (203) 447-1791 x364
3. Licensed Thermal Power (MWt): 2700
4. Nameplate Rating (Gross MWe): 1011 x 0.9 = 910
5. Design Electrical Rating (Net MWe): 870
6. Maximum Dependable Capacity (Gross MWe): 895
7. Maximum Dependable Capacity (Net MWe): 864
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

CAPACITY FACTORS CALCULATED WITH A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	6,575.0	41,783.0
12. Hours Reactor Was Critical	.0	4,276.7	29,355.4
13. Reactor Reserve Shutdown Hours	.0	.0	2,072.4
14. Hours Generator On-Line	.0	4,183.6	28,003.8
15. Unit Reserve Shutdown Hours	.0	.0	335.4
16. Gross Thermal Energy (MWH)	0	11,020,276	68,718,904
17. Gross Electrical Energy (MWH)	0	3,614,072	22,198,717
18. Net Electrical Energy (MWH)	-2,858	3,468,395	21,248,653
19. Unit Service Factor	.0	53.6	67.0
20. Unit Availability Factor	.0	63.6	67.8
21. Unit Capacity Factor (MDC Net)	.0	61.1	61.7
22. Unit Capacity Factor (DER Net)	.0	60.6	60.5
23. Unit Forced Outage Rate	.0	23.6	23.8

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 10/18/80

UNIT SHUTDOWN/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
13	08/16/80	S	720.0	C	4				CYCLE 4 REFUEL OUTAGE CONTINUED THROUGHOUT THE REPORTING PERIOD.

SUMMARY

MILLSTONE 2 REMAINED SHUTDOWN CONTINUING REFUELING DURING SEPTEMBER.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

MONTICELLO

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Minnesota/Wright 30 miles NW of Minneapolis
2.	Electric Reliability Council	Mid-Continent Area Reliability Coordination Agreement
3.	Type of Reactor	BWR
4.	Power Level (MWT)	1670
5.	Design Electrical Rating (MWE Net)	545
6.	Date of Initial Criticality	December 10, 1970
7.	Date of Commercial Operation	June 30, 1971
8.	Condenser Cooling Method	Once Through with Cooling Tower
9.	Condenser Cooling Water Source	Mississippi River

Utility & Contractor Information

10.	Licensee	Northern States Power Co.
11.	Corporate Address	414 Nicollet Mall - 8th Floor Minneapolis, Minnesota 55401
12.	Corporate Contact	G. H. Neils (Oper.) L. O. Mayer (Lic.)
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	GE
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. F. HEISHMAN
18.	IE Section Chief	W. S. Little
19.	IE Resident Inspector	C. Brown
20.	Licensing Project Manager	K. Eccleston
21.	Docket Number	50-263
22.	License Number/Date of Issue	DPR-22 issued 9/8/70
23.	Public Document Room	Environmental Conservation Library Minneapolis Public Library 300 Nicollet Mall Minneapolis, Minnesota 55401

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

Operating routinely.

+LAST IE SITE INSPECTION DATE: July 1-31, 1980

+ INSPECTION REPORT NO: 80-15

SUMMARY

INSPECTION SUMMARY

+ Inspection on July 1-31, (80-15): Routine inspection of control room operations, review of logs, selected emergency systems, followup of Licensee Event Reports, IE Bulletin 80-17, organization and administration, training personnel, and information meeting with local officials. The inspection involved 87.5 inspector hours onsite by two NRC inspectors, including 30 inspector hours onsite during off-shifts. No items of noncompliance or deviations were identified in these areas.

REPORTS RECEIVED FROM LICENSEE

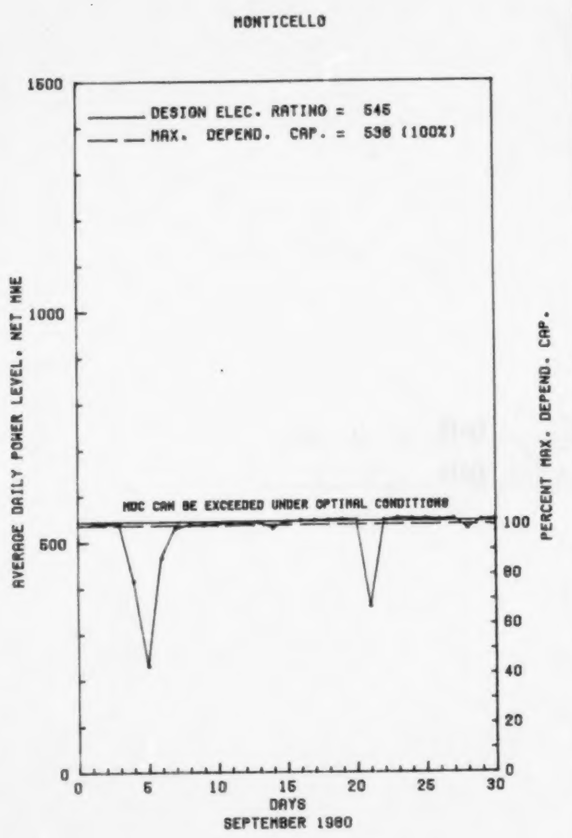
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-25/ 03L-0	07/29/80	08/28/80	HPCI valve failure to operate.
80-26/ 03L-0	08/17/80	09/16/80	"A" outboard MSIV inoperable.
80-27/ 03L-0	08/23/80	09/22/80	CRD 102 valve closed.
80-28/ 03L-0	08/25/80	09/19/80	RCIC steam leak.

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 11-4-80

REVIEWED BY NRR DL: Kenneth T. Eccleston **DATE:** 11-6-80

MONTICELLO

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-263
2. Utility Contact: A. L. Myrabo (612) 295-515'
3. Licensed Thermal Power (MWt): 1670
4. Nameplate Rating (Gross MWe): 632 x 0.9 = 569
5. Design Electrical Rating (Net MWe): 545
6. Maximum Dependable Capacity (Gross MWe): 564
7. Maximum Dependable Capacity (Net MWe): 536
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	6,575.0	81,120.0
12. Hours Reactor Was Critical	711.2	4,960.2	66,491.1
13. Reactor Reserve Shutdown Hours	.0	.0	940.7
14. Hours Generator On-Line	709.0	4,862.9	64,833.0
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,148,854	7,410,631	106,353,493
17. Gross Electrical Energy (MWH)	391,023	2,499,719	33,477,779
18. Net Electrical Energy (MWH)	374,213	2,379,215	32,011,419
19. Unit Service Factor	98.5	74.0	79.9
20. Unit Availability Factor	98.5	74.0	79.9
21. Unit Capacity Factor (MDC Net)	97.0	67.5	73.6
22. Unit Capacity Factor (DER Net)	95.4	66.4	72.4
23. Unit Forced Outage Rate	1.5	10.3	6.8

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): MARCH 28, 1981 - REFUELING OUTAGE - 56 DAYS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
13	09/04/80	F	11.0	A	3		HG	VALVEX	CONDENSATE DEMINERALIZER BACKWASH OPERATION VALVE FAILURE RESULTED IN FEEDWATER PUMP TRIP ON LOW SUCTION. REACTOR POWER WAS REDUCED RAPIDLY AND ONE RECIRCULATION PUMP WAS TRIPPED. DURING RECIRC PUMP SPEED INCREASE, A HIGH FLUX SCRAM WAS RECEIVED. VALVE REPAIRED.
14	09/21/80	S	.0	B	5	80-26	CD	VALVEX	REDUCED POWER TO 70% TO TEST MSIVS. OUTBOARD MSIV "A" FAILED TEST AND POWER WAS REDUCED TO 40% TO REPAIR. REPLACED AC AND DC SOLENOID VALVES ON "A" MSIV. REPLACED AC SOLENOID VALVE PLUNGER ON REMAINING OUTBOARD MSIVS.

SUMMARY

MONTICELLO OPERATED AT OR NEAR FULL POWER FOR SEPTEMBER, WITH THE EXCEPTION OF ONE OUTAGE ON THE 4TH FOR VALVE REPAIR AND ONE REDUCTION ON THE 21ST FOR VALVE MAINTENANCE.

LEGEND

[1] Type
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

NINE MILE POINT 1

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	New York/Oswego 8 miles NE of Oswego
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	1850
5.	Design Electrical Rating (MWE Net)	620
6.	Date of Initial Criticality	September 5, 1969
7.	Date of Commercial Operation	December 1969
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Lake Ontario

Utility & Contractor Information

10.	Licensee	Niagara Mohawk Power Corporation
11.	Corporate Address	300 Erie Boulevard West Syracuse, New York 13202
12.	Corporate Contact	R. Schneider, Vice President Electric Operations
13.	Architect/Engineer	Utility
14.	Nuclear Steam Supply System	GE
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Section Chief	H. B. KISTER
19.	IE Resident Inspector	W. BAUNACK (315) 342-4041 FTS: 264-3311 - Operator
20.	Licensing Project Manager	P. Polk
21.	Docket Number	50-220
22.	License Number/Date of Issue	DPR-63, 12/26/74
23.	Public Document Room	State University College of Oswego Penfield Library - Documents Oswego, NY 13126 (315)341-2323

INSPECTION STATUS

ENFORCEMENT STATUS

TECH SPEC 6.8.1 REQUIRES IN PART WRITTEN PROCEDURES...SHALL BE ESTABLISHED, IMPLEMENTED AND MAINTAINED THAT MEET OR EXCEED THE REQUIREMENTS OF ANSI N18.7-1972...." ANSI N18.7 PARAGRAPH 5.3.4.1(1) REQUIRES IN PART "...START UP PROCEDURES SHALL INCLUDE DETERMINATIONS THAT...VALVES SHOULD BE PROPERLY ALIGNED...CHECKOFF LIST SHOULD BE USED FOR THIS PURPOSE." CONTRARY TO THE ABOVE, MS 1-6 MAIN STEAM RELIEF VALVE BLOCKING VALVES WERE NOT INCLUDED ON THE VCOL NOR DOCUMENTED LOCKED OPEN.
(8005 INFRACTION)

FAILURE TO: 1) MAINTAIN CONTROLLED COPIES OF PROCEDURE IN CONTROL ROOM UP TO DATE, 2) HAVE STANDING ORDERS RECEIVED BY SORC WITHIN 7 DAYS OF IMPLEMENTATION, 3) MAINTAIN DOCUMENTATION OF TEMPORARY CHANGES, AND 4) IMPLEMENT PROCEDURE NI-ST-Q6, ADEQUATELY.
(8005 DEFICIENCY)

FAILURE TO WEAR THE NECESSARY PROTECTIVE CLOTHING INTO RADIATION/CONTAMINATION AREA AS REQUIRED BY RWP-9596 AND PROCEDURE RP-1 STEP 4.4.4.(C).
(8005 INFRACTION)

INADEQUATE ACCESS CONTROL TO VITAL AREA.
(8005 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS:

+ THE PLANT OPERATED AT 98% POWER ON FOUR RECIRCULATION PUMPS, EXCEPT FOR A SHUTDOWN ON SEPTEMBER 19 THRU SEPTEMBER 21 TO REPAIR FLOOR DRAIN LEAKAGE IN THE DRYWELL. REPRESENTATIVE STACK AND OFF-GAS RELEASE RATES ARE 10 AND 20,000 MICROCURIES PER SECOND, RESPECTIVELY.

+ LAST IE SITE INSPECTION DATE: 9/22-26/80

+ INSPECTION REPORT NO: 50-220/80-12

SUMMARY

INSPECTION SUMMARY

+ 50-220/80-05 - MAY 19-23: ROUTINE, UNANNOUNCED INSPECTION BY TWO REGION BASED INSPECTORS (65 HRS) OF LICENSEE ACTION ON PREVIOUS INSPECTION FINDINGS; ADMINISTRATIVE CONTROLS FOR FACILITY PROCEDURES; CONFORMANCE TO TECHNICAL SPECIFICATIONS; TEMPORARY AND PERMANENT CHANGES IN CONFORMANCE TO TECHNICAL SPECIFICATIONS AND LICENSEE PROCEDURES; CHANGES IN PROCEDURE TO 10 CFR 50.59(A) AND (B) REQUIREMENTS; CHECKLISTS AND RELATED FORMS FOR CURRENCY TO LATEST CHANGES; AND PLANT TOUR AND CONTROL ROOM OBSERVATIONS. SIX ITEMS OF NONCOMPLIANCE WERE IDENTIFIED: FAILURE TO MAINTAIN WRITTEN PROCEDURES; INADEQUATE ACCESS CONTROL TO A VITAL AREA; FAILURE TO WEAR NECESSARY PROTECTIVE CLOTHING INTO CONTAINMENT AREA; FAILURE TO PROPERLY IMPLEMENT AND HAVE SORC REVIEW OF STANDING ORDERS; FAILURE TO MAINTAIN CONTROL ROOM PROCEDURES UPDATED; FAILURE TO DOCUMENT TEMPORARY CHANGES.

+ 50-220/80-07 - JUN 25 - AUG 15: ROUTINE INSPECTION BY TWO RESIDENT INSPECTORS (109 HRS) OF LICENSEE ACTION ON PREVIOUS INSPECTION FINDINGS; OPERATIONAL SAFETY VERIFICATION; PLANT TOURS; OBSERVATION OF PHYSICAL SECURITY; WITNESSING OF SURVEILLANCE TESTS; LICENSEE ACTIONS TAKEN IN RESPONSE TO IE BULLETIN 80-17; VERIFYING LICENSE APPLICATION SUBMITTALS FOR LICENSE TRAINING STAFF PERSONNEL; LOCATION OF CERTAIN LOAD CENTERS; AND REVIEW OF PERIODIC REPORTS. NO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

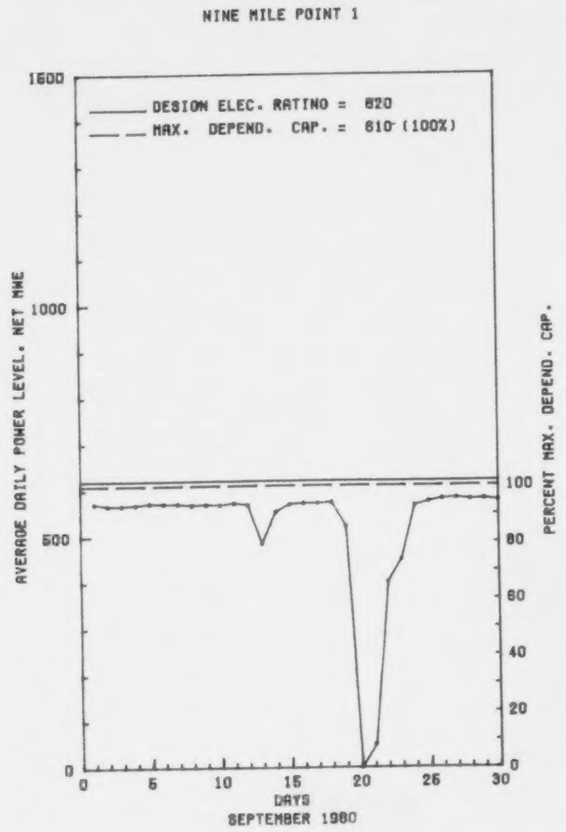
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE			

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 11-4-80

REVIEWED BY NRR DL: Philip J. Polk **DATE:** 11/6/80

NINE MILE POINT 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-220
2. Utility Contact: J. HALLENBECK (315) 343-2110 X1312
3. Licensed Thermal Power (MWt): 1850
4. Nameplate Rating (Gross MWe): 755 x 0.85 = 642
5. Design Electrical Rating (Net MWe): 620
6. Maximum Dependable Capacity (Gross MWe): 630
7. Maximum Dependable Capacity (Net MWe): 610
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): 598
10. Reasons for Restrictions, If Any: DERATE TO 98% CORE THERMAL POWER FOR SRI TIME FACTOR.

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	6,575.0	95,687.0
12. Hours Reactor Was Critical	693.0	6,027.7	71,380.4
13. Reactor Reserve Shutdown Hours	.0	.0	1,204.2
14. Hours Generator On-Line	677.4	5,926.1	68,736.6
15. Unit Reserve Shutdown Hours	.0	.0	20.2
16. Gross Thermal Energy (MWH)	1,194,261	10,373,380	112,116,779
17. Gross Electrical Energy (MWH)	387,558	3,436,900	36,946,410
18. Net Electrical Energy (MWH)	374,755	3,325,662	35,775,427
19. Unit Service Factor	94.1	90.1	71.8
20. Unit Availability Factor	94.1	90.1	71.9
21. Unit Capacity Factor (MDC Net)	85.3	82.9	61.3
22. Unit Capacity Factor (DER Net)	84.0	81.6	60.3
23. Unit Forced Outage Rate	5.9	6.4	9.1

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-19	09/19/80	F	42.6	A	1				SHUTDOWN FOR DRY WELL HIGH LEAKAGE (SHUTDOWN COOLING I.V. PACKING FAILURE).

SUMMARY

NINE MILE POINT OPERATED NORMALLY FOR SEPTEMBER WITH ONE OUTAGE ON THE 19TH FOR DRYWELL HIGH LEAKAGE.

LEGEND

[1] Type:
- F Forced
- S Scheduled

[2] Reason:
- A Equipment Failure
- B Maintenance or Test
- C Refueling
- D Regulatory Restriction
- E Operator Training & License Examination
- F Administrative
- G Operational Error
- H Other

[3] Method:
- 1 Manual
- 2 Manual Scram
- 3 Automatic Scram
- 4 Continuations
- 5 Load Reduction
- 9 Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source

NORTH ANNA 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Louisa County, Virginia 40 Miles NW of Richmond, Virginia
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	PWR
4.	Licensed Power Level (MWT)	2775
5.	Design Electrical Rating (MWE Net)	907
6.	Date of Initial Criticality	April 5, 1978
7.	Date of Commercial Operation	June 6, 1978
8.	Condenser Cooling Method	Once through
9.	Condenser Cooling Water Source	Lake Anne

Utility & Contractor Information

10.	Licensee	Virginia Electric & Power Co.
11.	Corporate Address	P. O. Box 26666 Richmond, Virginia 23261
12.	Corporate Contact	J. H. Ferguson Exec V.P. - Power
13.	Architect/Engineer	Stone & Webster
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. Lewis
18.	IE Section Chief	P. J. Kellog
19.	IE Resident Inspector	E. H. Webster (703) 894-5421
20.	Licensing Project Manager	L. Engle
21.	Docket Number	50-338
22.	License Number/Date of Issue	NPF-4, 11/26/77
23.	Public Document Room	Alderman Library/Manuscripts Dept. Univ. of Virginia/Charlottesville VA 22901 & Louisa County Courthouse, Louisa, VA 23093

INSPECTION STATUS

ENFORCEMENT STATUS

AS REQUIRED BY 10 CFR 20.103A(3) THE LICENSEE SHALL USE SUITABLE MEASUREMENTS OF CONCENTRATIONS OF RADIOACTIVE MATERIAL IN AIR FOR DETECTING AND EVALUATING AIRBORNE RADIOACTIVITY IN RESTRICTED AREAS. 10 CFR 20.401C(2)(I) STATES THAT RECORDS OF THE RESULTS OF SURVEYS TO DETERMINE COMPLIANCE WITH 20.103(A) SHALL BE MAINTAINED UNTIL THE COMMISSION AUTHORIZES THEIR DISPOSAL. CONTRARY TO THE ABOVE, THE RECORDS OF AIR SAMPLES TAKEN TO DETERMINE THE RESPIRATORY PROTECTION REQUIREMENTS FOR SEVERAL JOBS PERFORMED IN 1980 WERE NOT MAINTAINED.
(8022 DEFICIENCY)

AS REQUIRED BY TECHNICAL SPECIFICATION 6.4.1, A RETRAINING PROGRAM FOR THE FACILITY STAFF SHALL BE MAINTAINED AND SHALL MEET OR EXCEED THE REQUIREMENTS AND RECOMMENDATIONS OF SECTION 5.5 OF ANSI N18.1-1971. SECTION 5.5 OF ANSI N18.1 STATES THAT A TRAINING PROGRAM SHALL BE ESTABLISHED WHICH MAINTAINS THE PROFICIENCY OF THE OPERATING ORGANIZATION. CONTRARY TO THE ABOVE, THE LICENSEE DID NOT HAVE A FORMAL RETRAINING PROGRAM WHICH MAINTAINED THE PROFICIENCY OF THE HEALTH PHYSICS TECHNICIANS.
(8022 INFRACTION)

LICENSE NO. NPF-7 TECHNICAL SPECIFICATION 3.3.1.1 TABLE 3.3-1 FOR STEAM/FEEDWATER FLOW MISMATCH NOTE 7 REQUIRES THAT IF THE NUMBER OF OPERABLE CHANNELS, IS ONE LESS THAN THE TOTAL NUMBER OF CHANNELS, STARTUP AND POWER OPERATIONS MAY PROCEED UNTIL THE NEXT REQUIRED CHANNEL FUNCTIONAL TEST PROVIDED THE INOPERABLE CHANNEL IS PLACED IN THE TRIPPED CONDITION WITHIN ONE HOUR. ADDITIONALLY, TECHNICAL SPECIFICATION 3.0.3 STATES THAT IF A LIMITING CONDITION FOR OPERATION AND/OR ASSOCIATED ACTION REQUIREMENTS CANNOT BE SATISFIED BECAUSE OF CIRCUMSTANCES IN EXCESS OF THOSE ADDRESSED IN THE SPECIFICATION, THE UNIT SHALL BE PLACED IN AT LEAST HOT STANDBY WITHIN ONE HOUR AND HOT SHUTDOWN WITHIN THE NEXT SIX HOURS.
(8028 INFRACTION)

LICENSE NO. NPF-7 TECHNICAL SPECIFICATION 6.8.1.A REQUIRES THAT PROCEDURES BE DEVELOPED, IMPLEMENTED AND MAINTAINED FOR THE PROCEDURES RECOMMENDED IN APPENDIX A OF REGULATORY GUIDE 1.33, REVISION 2, FEBRUARY 1978. SECTION 3 OF APPENDIX A DISCUSSES PROCEDURES FOR STARTUP, OPERATIONS AND SHUTDOWN OF SAFETY RELATED SYSTEMS. CONTRARY TO THE ABOVE, 2-OP-1A "PRE STARTUP CHECKOFF LIST" HAD NOT BEEN REVISED TO REQUIRE THE CHECKING OF INSTRUMENT STOP VALVES IN ORDER TO INSURE REACTOR PROTECTION SYSTEM INSTRUMENTATION WAS IN SERVICE PRIOR TO ENTERING THE MODES WHERE IT WOULD BE REQUIRED. SPECIFICALLY, ON JUNE 19, 1980, WHILE IN MODE 2, FLOW TRANSMITTERS FT-2476, FT-2477, FT-2487, FT-2496, AND FT-2497 WERE FOUND TO BE ISOLATED.
(8028 INFRACTION)

AS REQUIRED BY 10 CFR 20.103A(3) THE LICENSEE SHALL USE SUITABLE MEASUREMENTS OF CONCENTRATIONS OF RADIOACTIVE MATERIAL IN AIR FOR DETECTING AND EVALUATING AIRBORNE RADIOACTIVITY IN RESTRICTED AREAS. 10 CFR 20.401C(2)(I) STATES THAT RECORDS OF THE RESULTS OF SURVEYS TO DETERMINE COMPLIANCE WITH 20.103(A) SHALL BE MAINTAINED UNTIL THE COMMISSION AUTHORIZES THEIR DISPOSAL. CONTRARY TO THE ABOVE, THE RECORDS OF AIR SAMPLES TAKEN TO DETERMINE THE RESPIRATORY PROTECTION REQUIREMENTS FOR SEVERAL JOBS PERFORMED IN 1980 WERE NOT MAINTAINED.
(8021 DEFICIENCY)

AS REQUIRED BY TECHNICAL SPECIFICATION 6.4.1, A RETRAINING PROGRAM FOR THE FACILITY STAFF SHALL BE MAINTAINED AND SHALL MEET OR EXCEED THE REQUIREMENTS AND RECOMMENDATIONS OF SECTION 5.5 OF ANSI N18.1-1971. SECTION 5.5 OF ANSI N18.1 STATES THAT A TRAINING PROGRAM SHALL BE ESTABLISHED WHICH MAINTAINS THE PROFICIENCY OF THE OPERATING ORGANIZATION. CONTRARY TO THE ABOVE, THE LICENSEE DID NOT HAVE A FORMAL RETRAINING PROGRAM WHICH MAINTAINED THE PROFICIENCY OF THE HEALTH PHYSICS TECHNICIANS.
(8021 INFRACTION)

LICENSE NO. NPF-4 TECHNICAL SPECIFICATION 3.3.1.1 TABLE 3.3-1 FOR STEAM GENERATOR WATER LEVEL LOW-LOW AND LOW STEAM GENERATOR WATER LEVEL NOTE 7 REQUIRES THAT IF THE NUMBER OF OPERABLE CHANNELS IS ONE LESS THAN THE TOTAL NUMBER OF CHANNELS, STARTUP AND POWER OPERATIONS MAY PROCEED UNTIL THE NEXT REQUIRED CHANNEL FUNCTIONAL TEST PROVIDED THE INOPERABLE CHANNEL IS PLACED IN THE TRIPPED CONDITION WITHIN ONE HOUR. CONTRARY TO THE ABOVE, ON JUNE 12, 1980, WHILE OPERATING IN MODE 1 THE INOPERABLE STEAM GENERATOR LEVEL CHANNEL WAS NOT PLACED IN THE TRIPPED CONDITION WITHIN ONE HOUR.
(8029 INFRACTION)

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-060/ 03L-0	07/05/83	07/30/80	SAFEGUARDS AREA VENTILATION SYSTEM FILTER I-HV-FL-3A INOPERABLE T.S. 3.7.8.1.
80-063/ 03L-0	08/08/80	08/27/80	VENTURI FOULING IN FEEDWATER LINES CAUSED 4% ERROR IN FEEDWATER FLOW SIGNAL.
80-064/ 03L-0	07/19/80	08/08/80	CONTAINMENT BULK AVERAGE TEMPERATURE EXCEEDED T.S. 3.6.1.5 LIMIT OF 105 DEGREES.
80-066/ 03L-0	07/20/80	08/08/80	CONTAINMENT BULK AVERAGE TEMPERATURE EXCEEDED T.S. 3.6.1.5 LIMIT OF 105 DEGREES.
80-067/ 03L-0	07/24/80	08/14/80	PT-57.7 MITIGATION SYSTEM LEAK CHECK NOT PERFORMED WITHIN 92.

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

100% OPERATION.

+ LAST IE SITE INSPECTION DATE: AUGUST 18, 1980

+ INSPECTION REPORT NO: 50-338/80-17

SUMMARY

INSPECTION SUMMARY

+ INSPECTION AUGUST 5-8 AND 15-18 (80-17): THIS SPECIAL, UNANNOUNCED INSPECTION INVOLVED 68 INSPECTOR-HOURS ON SITE IN THE AREAS OF IMPLEMENTATION OF THE NEW NORTH ANNA EMERGENCY PLAN, INCLUDING COORDINATION WITH OFFSITE SUPPORT AGENCIES, EMERGENCY FACILITIES, EQUIPMENT AND PROCEDURES, EMERGENCY TRAINING, EMERGENCY TESTS AND DRILLS, FOLLOWUP ON PREVIOUS INSPECTION FINDINGS AND IE BULLETIN 79-18. OF THE SIX AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

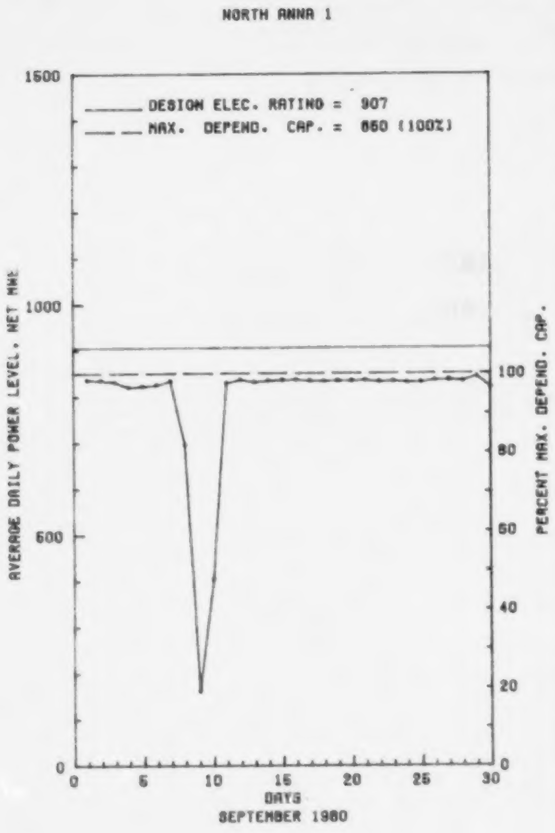
INSPECTION MAY 5-16 (80-21): THIS SPECIAL ANNOUNCED INSPECTION INVOLVED 158 INSPECTOR-HOURS ON SITE IN THE AREA OF HEALTH PHYSICS APPRAISAL INCLUDING RADIATION PROTECTION ORGANIZATION AND MANAGEMENT, PERSONNEL SELECTION, QUALIFICATION AND TRAINING, EXPOSURE CONTROLS, ALARA PROGRAM, RADIOACTIVE WASTE MANAGEMENT, FACILITIES AND EQUIPMENT, AND EMERGENCY RESPONSE CAPABILITIES. OF THE SEVEN AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN FIVE AREAS; TWO ITEMS OF NONCOMPLIANCE WERE FOUND IN TWO AREAS (INFRACTION - FAILURE TO HAVE A RETRAINING PROGRAM FOR HEALTH PHYSICS TECHNICIANS; DEFICIENCY - FAILURE TO HAVE RECORDS OF AIRBORNE RADIOACTIVITY SURVEYS).

REVIEWED BY OMPA LOEB: PE Bote **DATE:** 11/5/80

REVIEWED BY NRR DL: S Eng **DATE:** 11/6/80

NORTH ANNA 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-338
2. Utility Contact: W. R. Madison (703) 894-5151
3. Licensed Thermal Power (MWt): 2775
4. Nameplate Rating (Gross MWe): 1089 x 0.9 = 980
5. Design Electrical Rating (Net MWe): 907
6. Maximum Dependable Capacity (Gross MWe): 890
7. Maximum Dependable Capacity (Net MWe): 850
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	6,575.0	20,352.0
12. Hours Reactor Was Critical	713.9	5,693.7	15,856.9
13. Reactor Reserve Shutdown Hours	6.1	61.7	210.0
14. Hours Generator On-Line	713.4	5,494.1	15,547.2
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,887,261	14,045,342	40,325,255
17. Gross Electrical Energy (MWH)	605,270	4,411,668	12,758,017
18. Net Electrical Energy (MWH)	570,284	4,147,688	12,001,134
19. Unit Service Factor	99.1	83.6	76.4
20. Unit Availability Factor	99.1	83.6	76.4
21. Unit Capacity Factor (MDC Net)	93.2	74.2	69.4
22. Unit Capacity Factor (DER Net)	87.3	69.6	65.0
23. Unit Forced Outage Rate	.9	9.1	10.0

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
REFUELING - DECEMBER 31, 1980 - SIX WEEKS
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-21	09/08/80	F	6.5	A	3	80-86	EB	GENERA	REACTOR TRIP FROM TURBINE TRIP CAUSED BY "C" S/G LO LO LEVEL WHEN THE OUTPUT BREAKER FROM INVERTER OPENED CAUSING A LOSS OF POWER TO VITAL BUS IV. CHECKED FUSES, GROUNDS AND RECLOSED OUTPUT BREAKER FROM INVERTER.
80-22	09/08/80	F	.0	G	3				HI HI LEVEL "C" S/G OVER FILLED DURING UNIT STARTUP.
80-23	09/30/80	F	.1	A	3				REACTOR TRIP FROM TURBINE TRIP WHEN 6TH POINT HEATER LEVEL WENT ABOVE THE HIGH LEVEL TRIP POINT DUE TO A TUBE FAILURE. PLUGGED AFFECTED TUBES.

SUMMARY

NORTH ANNA 1 OPERATED AT NEAR FULL POWER WITH TWO OUTAGES CAUSED BY TURBINE TRIPS ON THE 8TH AND 30TH, AND ONE REDUCTION ON THE 8TH FROM OVERFILLING THE STEAM GENERATOR.

LEGEND

[1] Type:
F-Forced
S-Scheduled

[2] Reason:
A-Equipment Failure
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error
H-Other

[3] Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Continuations
5-Load Reduction
9-Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for License Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

NORTH ANNA 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Louisa County, VA 40 Miles NW of Richmond, VA
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	PWR
4.	Licensed Power Level (MWT)	2775
5.	Design Electrical Rating (MWE Net)	907
6.	Date of Initial Criticality	June 12, 1980
7.	Date of Commercial Operation	N/A
8.	Condenser Cooling Method	Once through
9.	Condenser Cooling Water Source	Lake Anna

Utility and Contractor Information

10.	Licensee	Virginia Electric & Power Co.
11.	Corporate Address	P.O. Box 26666 Richmond, VA 23261
12.	Corporate Contact	J. H. Ferguson, Exec. V.P. - Power
13.	Architect/Engineer	Stone & Webster
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. Lewis
18.	IE Section Chief	P. J. Kellog
19.	IE Resident Inspector	E. H. Webster (703) 894-5421
20.	Licensing Project Manager	L. Engle
21.	Docket Number	50-339
22.	License Number/Date of Issue	NPF-7, 8/21/80
23.	Public Document Room	Alderman Library/Manuscripts Dept. Univ. of Virginia/Charlottesville, VA 22901 & Louisa County Courthouse, Louisa, VA 23093

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

CONTINUE POWER ESCALATION TESTING

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

75% FULL POWER.

+ LAST IE SITE INSPECTION DATE: AUGUST 18, 1980

+ INSPECTION REPORT NO: 50-339/80-30

SUMMARY

INSPECTION SUMMARY

+ INSPECTION MAY 5-16 (80-22): THIS SPECIAL ANNOUNCED INSPECTION INVOLVED 157 INSPECTOR-HOURS ON SITE IN THE AREA OF HEALTH PHYSICS APPRAISAL INCLUDING RADIATION PROTECTION ORGANIZATION AND MANAGEMENT, PERSONNEL SELECTION, QUALIFICATION AND TRAINING, EXPOSURE CONTROLS, ALARA PROGRAM, RADIOACTIVE WASTE MANAGEMENT, FACILITIES AND EQUIPMENT, AND EMERGENCY RESPONSE CAPABILITIES. OF THE SEVEN AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN FIVE AREAS; TWO ITEMS OF NONCOMPLIANCE WERE FOUND IN TWO AREAS (INFRACTION - FAILURE TO HAVE A RETRAINING PROGRAM FOR HEALTH PHYSICS TECHNICIANS; DEFICIENCY - FAILURE TO HAVE RECORDS OF AIRBORNE RADIOACTIVITY SURVEYS).

INSPECTION AUGUST 5-8 AND 15-18 (80-30): THIS SPECIAL, UNANNOUNCED INSPECTION INVOLVED 67 INSPECTOR-HOURS ON SITE IN THE AREAS OF IMPLEMENTATION OF THE NEW NORTH ANNA EMERGENCY PLAN, INCLUDING COORDINATION WITH OFFSITE SUPPORT AGENCIES, EMERGENCY FACILITIES, EQUIPMENT AND PROCEDURES, EMERGENCY TRAINING, EMERGENCY TESTS AND DRILLS, FOLLOWUP ON PREVIOUS INSPECTION FINDINGS AND IE BULLETIN 79-18. OF THE SIX AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

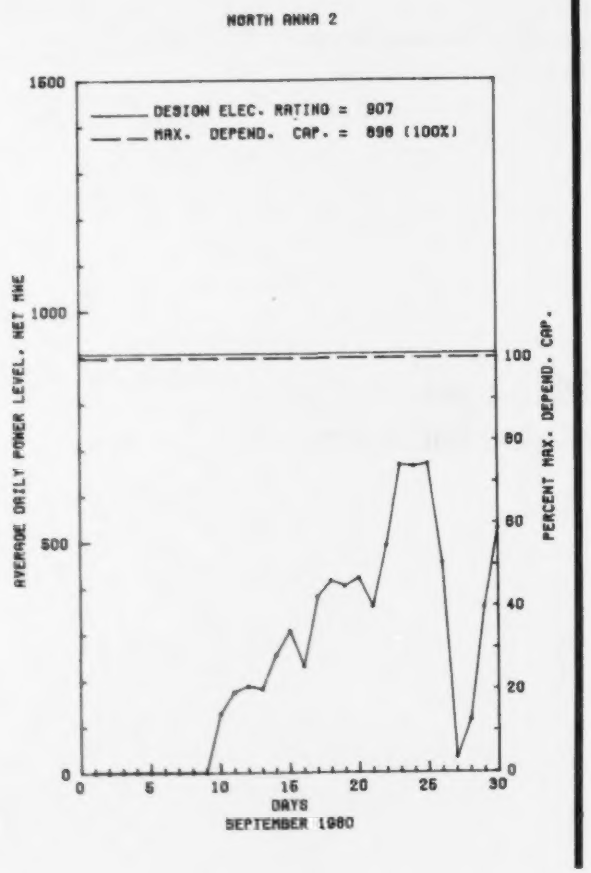
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-025/ 03L-0	07/06/80	07/30/80	CONTROL ROD F-14 INDICATION FAILED T.S. 3.1.3.2. SIGNAL MODULE REPLACED.
80-036/ 03L-0	07/17/80	08/14/80	TURBINE AUX. FEEDWATER PUMP LINED TO COMMON HEADER TO ALL 3 STEAM GENERATORS.
80-040/ 03L-0	07/23/80	08/14/80	CONTAINMENT BULK AVERAGE TEMPERATURE EXCEEDED T.S. 3.6.1.5 LIMIT OF 105 DEGREES.

REVIEWED BY OMPA LOEB: PEBode **DATE:** 11/5/80

REVIEWED BY NRR DL: ________ **DATE:** ________

NORTH ANNA 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-339
2. Utility Contact: W. A. WOODSMALL, III (703) 894-5151
3. Licensed Thermal Power (MWt): 2775
4. Nameplate Rating (Gross MWe): 947
5. Design Electrical Rating (Net MWe): 907
6. Maximum Dependable Capacity (Gross MWe): 928
7. Maximum Dependable Capacity (Net MWe): 898
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	4,392.0	4,352.0
12. Hours Reactor Was Critical	476.8	1,723.8	1,723.8
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	459.4	514.4	514.4
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	615,501	663,680	663,680
17. Gross Electrical Energy (MWH)	193,846	202,020	202,020
18. Net Electrical Energy (MWH)	177,547	184,109	184,109
19. Unit Service Factor			
20. Unit Availability Factor			
21. Unit Capacity Factor (MDC Net)			
22. Unit Capacity Factor (DER Net)			
23. Unit Forced Outage Rate			

NOT IN COMMERCIAL OPERATION

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): TMI & FIRE PROT MOD -11/1/80- 21 DAYS-MAINT -4/3/81- 11 DAYS
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause and Corrective Action to Prevent Recurrence
80-04	08/28/80	F	225.0	H	4				TURBINE TRIP/REACTOR TRIP WHEN GENERATOR BREAKER WAS OPENED. REPAIR CONCLUDED 09/10/80.
80-05	09/16/80	F	3.4	A	3				REACTOR TRIP/TURBINE TRIP DUE TO LOW VACUUM. VACUUM WAS LOST WHILE REMOVING COVER TO CONDENSATE PUMP SUCTION STRAINER FOR CLEANING BECAUSE A VALVE TO THE STRAINER WAS NOT FULLY CLOSED. VALVE AND CONDENSATE SUCTION STRAINER WERE CLEANED.
80-06	09/26/80	F	4.0	A	3				FEEDWATER PUMPS TRIPPED TRIPPING THE REACTOR. CLEANED PUMP SUCTION STRAINERS.
80-07	09/27/80	S	20.6	B	3				50% TEST REACTOR TRIP PER 2-SU-26
80-08	09/28/80	S	7.6	B	9				NO. 2 INTERCEPT RIGHT VALVE WAS INDICATING OPEN AND SHUT AT THE SAME TIME.

SUMMARY

NORTH ANNA 2 CAME ON-LINE FROM A TURBINE/REACTOR TRIP ON THE 10TH, AND OPERATED WITH FOUR ADDITIONAL OUTAGES IN SEPTEMBER FOR EQUIPMENT FAILURE AND MAINTENANCE.

LEGEND

[1]Type:
F--Forced
S--Scheduled

[2]Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & Licensee Examination
F--Administrative
G--Operational Error
H--Other

[3]Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4]Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5]Exhibit H:
Same source

OCONEE 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	South Carolina/Oconee 30 miles W of Greenville
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2568
5.	Design Electrical Rating (MWE Net)	887
6.	Date of Initial Criticality	April 19, 1973
7.	Date of Commercial Operation	July 15, 1973
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Keowee Lake

Utility & Contractor Information

10.	Licensee	Duke Power Company
11.	Corporate Address	422 S. Church Street Charlotte, North Carolina 28242
12.	Corporate Contact	W. O. Parker, Jr.
13.	Architect/Engineer	DPC and Bechtal (Containment Bldg.)
14.	Nuclear Steam Supply System	B&W
15.	Constructor	DPC

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R.C. Lewis
18.	IE Section Chief	R. MARTIN
19.	IE Resident Inspector	F. Jape (803) 882-6927
20.	Licensing Project Manager	M. B. Fairtile
21.	Docket Number	50-269
22.	License Number/Date of Issue	DPR-38, 2/6/73
23.	Public Document Room	Oconee County Library 201 S. Spring Street Walhalla, South Carolina 29691

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO THE REQUIREMENTS OF 10CFR50, APPENDIX B CRITERION VIII AS IMPLEMENTED IN SECTION 17.2.8 OF DUKE TOPICAL REPURT 1A TRACEABILITY AND ACCOUNTABILITY WAS NOT MAINTAINED ON MATERIAL USED IN REACTOR COOLANT SYSTEM MECHANICAL JOINTS. NOR IS THE REQUIREMENT ADDRESSED IN LICENSEE PROCEDURES OR DIRECTIVES.
(8028 INFRACTION)

CONTRARY TO THE REQUIREMENTS OF TECHNICAL SPECIFICATION 6.4.1, A CURRENT, WRITTEN, APPROVED PROCEDURE WAS NOT EMPLOYED IN THE TRANSPORT OF LETDOWN FILTER CASK; THE TRANSPORT OF WHICH RESULTED IN A CONTAMINATED MATERIAL SPLIT.
(8028 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

ROUTINE OPERATION.

LAST IE SITE INSPECTION DATE: SEPTEMBER 23-26, 1980

INSPECTION REPORT NO: 50-269/80-33

SUMMARY

INSPECTION SUMMARY

INSPECTION JULY 1-31 (80-28): THIS ROUTINE INSPECTION INVOLVED 74 RESIDENT INSPECTOR-HOURS ON SITE IN THE AREAS OF PLANT OPERATION, SURVEILLANCE TESTING, TEST AND MEASURING EQUIPMENT, MAINTENANCE PROGRAM, PHYSICAL SECURITY, PLANT MODIFICATION TEST WITNESSING, RADIOACTIVE WASTE TRANSPORT, INSTRUCTOR LICENSING, LER FOLLOWUP, SPENT FUEL SHIPMENT, AND OFFSITE CONTAMINATION. OF THE 11 AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN 9 AREAS; 2 ITEMS OF NONCOMPLIANCE WERE FOUND IN 2 AREAS (INFRACTION: FAILURE TO MAINTAIN TRACEABILITY AND ACCOUNTABILITY OF QA MATERIAL USED ON SAFETY-RELATED SYSTEM; INFRACTION - FAILURE TO EMPLOY PROCEDURE WHEN MOVING RADIOACTIVE WASTE).

INSPECTION AUGUST 25-28 (80-29): INCLUDED REVIEW OF THE LICENSEE'S CORRECTIVE ACTIONS TO PRIOR ITEM OF NONCOMPLIANCE 79-24-01. ALSO INSPECTION OF SECURITY ORGANIZATION, BARRIERS, ACCESS CONTROLS, ALARM STATIONS, DETECTION AIDS, COMMUNICATIONS, COMPENSATORY MEASURES, POWER SUPPLY AND I&E INFORMATION NOTICE NO. 80-18. THE INSPECTION INVOLVED 9 INSPECTOR-HOURS ON SITE BY ONE NRC INSPECTOR. THE INSPECTION WAS BEGUN DURING A REGULAR PERIOD; 3 INSPECTION HOURS WERE ACCOMPLISHED DURING OFFSHIFT PERIODS. THE LICENSEE WAS FOUND TO BE IN COMPLIANCE WITH NRC REQUIREMENTS WITHIN ALL THE AREAS EXAMINED DURING THE INSPECTION. CORRECTIVE ACTION TO 79-24-01 WAS FOUND ACCEPTABLE.

INSPECTION AUGUST 1-31 (80-30): THIS ROUTINE INSPECTION INVOLVED 58 (RESIDENT) INSPECTOR-HOURS ON SITE IN THE AREAS OF PLANT OPERATIONS, SURVEILLANCE TESTING, INSTRUMENT CALIBRATION, PHYSICAL SECURITY, LER REVIEW, CONTAMINATED TENDON GALLERIES, HYDROGEN GENERATION INSIDE CONTAINMENT, AND LOAD CENTER LOCATION. OF THE EIGHT AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

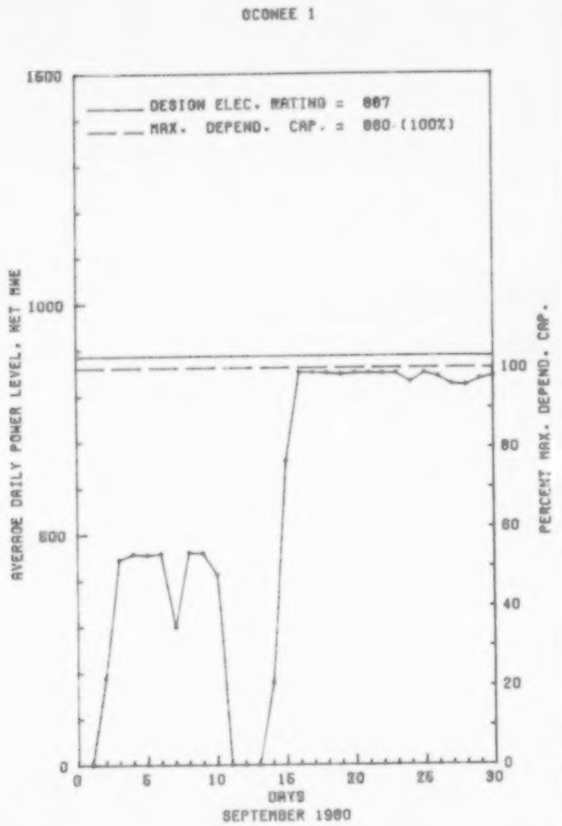
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-023/03L-0	07/10/80	07/23/80	REACTIVITY ANOMALY OF GREATER THAN 1% DELTA K/K.
80-024/03L-0	07/31/80	08/29/80	AUTOMATIC INITIATION OF HIGH PRESSURE SERVICE WATER PUMPS WAS DISABLED.
80-025/03L-0	08/07/80	09/05/80	KEOWEE HYDRO UNIT 2 WAS INOPERABLE DUE TO DIRTY CONTACTS.
80-026/03L-0	08/11/80	09/10/80	HIGH PRESSURE SERVICE WATER PUMP "B" INOPERABLE DUE TO COOLER LEAK.

REVIEWED BY OMPA LOEB: PE Bole **DATE:** 11/4/80

REVIEWED BY NRR DL: MB Fairtile **DATE:** 11/5/80

OCONEE 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-269
2. Utility Contact: J. A. Reavis (704) 373-8552
3. Licensed Thermal Power (MWt): 2568
4. Nameplate Rating (Gross MWe): 1038 x 0.9 = 934
5. Design Electrical Rating (Net MWe): 887
6. Maximum Dependable Capacity (Gross MWe): 899
7. Maximum Dependable Capacity (Net MWe): 860

CAPACITY FACTORS CALCULATED WITH A WEIGHTED AVERAGE

8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	6,575.0	63,216.0
12. Hours Reactor Was Critical	622.1	4,564.0	45,078.7
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	597.5	4,436.4	42,381.5
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,265,125	9,643,779	98,861,898
17. Gross Electrical Energy (MWH)	435,040	3,409,110	34,323,410
18. Net Electrical Energy (MWH)	408,066	3,226,444	32,457,963
19. Unit Service Factor	83.0	67.4	67.0
20. Unit Availability Factor	83.0	67.4	67.1
21. Unit Capacity Factor (MDC Net)	65.9	57.1	59.5
22. Unit Capacity Factor (DER Net)	61.9	55.3	58.0
23. Unit Forced Outage Rate	17.0	14.5	17.6

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
TMI-RELATED MODIFICATIONS - NOVEMBER 16 - 3 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
7	08/20/80	F	35.4	A	4		CH	HTEXCH	OUTAGE DUE TO FEEDWATER HEATER 1B1 TUBE RUPTURE CONTINUES.
8-P	09/02/80	F	.0	A	5		RB	CRDRVE	CONTROL ROD #8 GP. #7 DROPPED CAUSING A POWER REDUCTION.
8	09/07/80	F	3.8	A	3		HC	VALVEX	UNIT TRIPPED DUE TO (MSDT) MOISTURE SEPARATOR DRAIN TANK HIGH LEVEL. AIR LINE BROKE ALLOWING VALVE 1HD-59 TO FAIL CLOSED.
9-P	09/07/80	F	.0	A	5		RB	CRDVE	CONTROL ROD #8 GP. #7 REMAINS DROPPED FORCING OPERATION AT REDUCED POWER.
9	09/11/80	F	83.5	A	1		RB	CRDVE	UNIT SHUTDOWN FOR ROD #8 GP. #7 STATOR REPLACEMENT.
10-P	09/15/80	F	.0	H	5		HA	FILTER	HOLD IN POWER DUE TO POWDEX LIMITATION.

SUMMARY

OCONEE 1 OPERATED NORMALLY DURING SEPTEMBER, WITH THREE OUTAGES AND THREE REDUCTIONS LISTED IN DETAIL ABOVE.

LEGEND

[1] Type:
F-Forced
S-Scheduled

[2] Reason:
A-Equipment Failure
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error
H-Other

[3] Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Continuations
5-Load Reduction
9-Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

OCONEE 2

FACILITY DATA

Facility Description

1. Location (state/county) (nearest population center)	South Carolina/Oconee 30 miles W of Greenville
2. Electric Reliability Council	Southeastern Electric Reliability Council
3. Type of Reactor	PWR
4. Power Level (MWT)	2568
5. Design Electrical Rating (MWE Net)	887
6. Date of Initial Criticality	November 11, 1973
7. Date of Commercial Operation	September 9, 1974
8. Condenser Cooling Method	Once Through
9. Condenser Cooling Water Source	Keowee Lake

Utility & Contractor Information

10. Licensee	Duke Power Company
11. Corporate Address	422 S. Church Street Charlotte, North Carolina 28242
12. Corporate Contact	W. O. Parker, Jr.
13. Architect/Engineer	DPC/Bechtel
14. Nuclear Steam Supply System	B&W
15. Constructor	DPC

Regulatory Information

16. IE Region responsible	II
17. IE Branch Chief	R.C. Lewis
18. IE Section Chief	R. MARTIN
19. IE Resident Inspector	F. Jape (803) 882-6927
20. Licensing Project Manager	M. B. Fairtile
21. Docket Number	50-270
22. License Number/Date of Issue	DPR-47, 10/6/73
23. Public Document Room	Oconee County Library 201 S. Spring Street Walhalla, South Carolina 29691

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE

FACILITY ITEMS (PLANS AND PROCEDURES):

FIVE WEEK MAINTENANCE OUTAGE SCHEDULED TO BEGIN IN MID OCTOBER, 1980.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

ROUTINE OPERATION.

LAST IE SITE INSPECTION DATE: SEPTEMBER 23-26, 1980

INSPECTION REPORT NO: 50-270/80-29

SUMMARY

INSPECTION SUMMARY

INSPECTION JULY 1-31 (80-24): THIS ROUTINE INSPECTION INVOLVED 74 RESIDENT INSPECTOR-HOURS ON SITE IN THE AREAS OF PLANT OPERATION, SURVEILLANCE TESTING, TEST AND MEASURING EQUIPMENT, MAINTENANCE PROGRAM, PHYSICAL SECURITY, PLANT MODIFICATION TEST WITNESSING, RADIOACTIVE WASTE TRANSPORT, INSTRUCTOR LICENSING, LER FOLLOWUP, SPENT FUEL SHIPMENT, AND OFFSITE CONTAMINATION. OF THE 11 AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN 9 AREAS; 2 ITEMS OF NONCOMPLIANCE WERE FOUND IN 2 AREAS (INFRACTION - FAILURE TO MAINTAIN TRACEABILITY AND ACCOUNTABILITY OF QA MATERIAL USED ON SAFETY-RELATED SYSTEM; INFRACTION - FAILURE TO EMPLOY PROCEDURE WHEN MOVING RADIOACTIVE WASTE).

INSPECTION AUGUST 25-28 (80-25): INCLUDED REVIEW OF THE LICENSEE'S CORRECTIVE ACTIONS TO PRIOR ITEM OF NONCOMPLIANCE 79-24-01, ALSO INSPECTION OF SECURITY ORGANIZATION, BARRIERS, ACCESS CONTROLS, ALARM STATIONS, DETECTION AIDS, COMMUNICATIONS, COMPENSATORY MEASURES, POWER SUPPLY AND I&E INFORMATION NOTICE NO. 80-18. THE INSPECTION INVOLVED 9 INSPECTOR-HOURS ON SITE BY ONE NRC INSPECTOR. THE INSPECTION WAS BEGUN DURING A REGULAR PERIOD; 3 INSPECTION HOURS WERE ACCOMPLISHED DURING OFFSHIFT PERIODS. THE LICENSEE WAS FOUND TO BE IN COMPLIANCE WITH NRC REQUIREMENTS WITHIN ALL THE AREAS EXAMINED DURING THE INSPECTION. CORRECTIVE ACTION TO 79-24-01 WAS FOUND ACCEPTABLE.

INSPECTION AUGUST 1-31 (80-26): THIS ROUTINE INSPECTION INVOLVED 58 (RESIDENT) INSPECTOR-HOURS ON SITE IN THE AREAS OF PLANT OPERATIONS, SURVEILLANCE TESTING, INSTRUMENT CALIBRATION, PHYSICAL SECURITY, LER REVIEW, CONTAMINATED TENDON GALLERIES, HYDROGEN GENERATION INSIDE CONTAINMENT, AND LOAD CENTER LOCATION. OF THE EIGHT AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-011/ 03L-0	07/28/80	08/25/80	NUMBER 2B HIGH PRESSURE INJECTION PUMP WAS INOPERABLE.
80-012/ 03L-0	08/14/80	08/29/80	CONCENTRATED BORIC ACID STORAGE TANK PUMP WAS INOPERABLE FOR FOUR HOURS.
80-013/ 03L-0	08/05/80	09/04/80	THE CONCENTRATED BORIC ACID STORAGE TANK WAS INOPERABLE DUE TO A CLOGGED STRAINER.

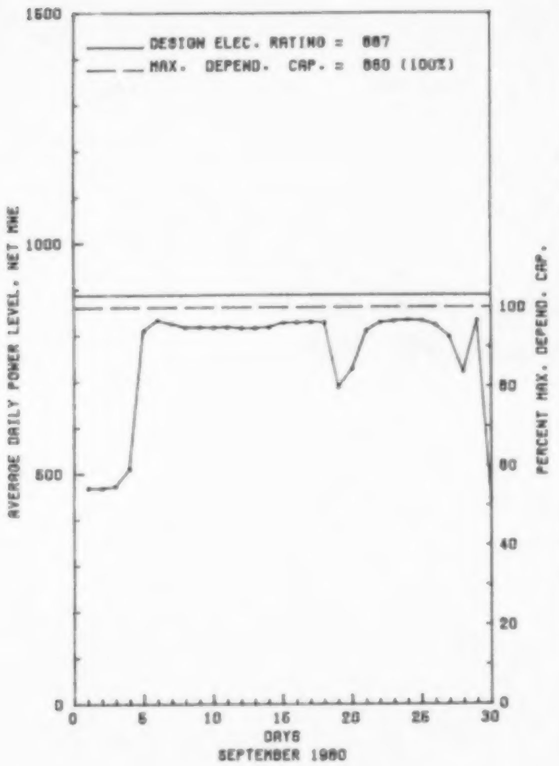
REVIEWED BY OMPA LOEB: K Bobe **DATE:** 11/4/80

REVIEWED BY NRR DL: MB Fairtile **DATE:** 11/5/80

OCONEE 2

AVERAGE DAILY POWER LEVEL (MWe)

OCONEE 2



OPERATING STATUS

1. Docket Number: 50-270
2. Utility Contact: J. A. Reavis (704) 373-8552
3. Licensed Thermal Power (MWt): 2568
4. Nameplate Rating (Gross MWe): 1038 x 0.9 = 934
5. Design Electrical Rating (Net MWe): 887
6. Maximum Dependable Capacity (Gross MWe): 899
7. Maximum Dependable Capacity (Net MWe): 860
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

CAPACITY FACTORS CALCULATED WITH A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	6,575.0	53,136.0
12. Hours Reactor Was Critical	718.7	3,962.7	37,558.7
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	716.2	3,860.4	36,634.5
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,683,265	8,194,902	86,242,306
17. Gross Electrical Energy (MWH)	568,930	2,778,210	29,292,566
18. Net Electrical Energy (MWH)	541,444	2,622,678	27,786,436
19. Unit Service Factor	99.5	58.7	68.9
20. Unit Availability Factor	99.5	58.7	68.9
21. Unit Capacity Factor (MDC Net)	87.4	46.4	60.8
22. Unit Capacity Factor (DER Net)	84.8	45.0	59.0
23. Unit Forced Outage Rate	.5	2.8	18.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): TMI-RELATED MODIFICATIONS - OCTOBER 24 - 3 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
12-P	09/01/80	F	.0	D	5		SF	PUMPXX	POWER REDUCTION CONTINUES PER TECH SPEC FOR TWO HPI PUMP OPERATION. MAINTENANCE ON 2 "B" HPI PUMP.
13-P	09/19/80	F	.0	D	5		SF	PUMPXX	POWER REDUCTION PER TECH SPEC FOR TWO HPI PUMP OPERATION. MAINTENANCE ON 2B HPI PUMP EXCEEDING THE 72 HOUR TIME PERIOD.
14-P	09/20/80	F	.0	B	5		ZZ	ZZZZZZ	HOLD IN POWER AT 90% FOR NI CALIBRATION.
3	09/30/80	F	3.8	F	3		HA	TURBIN	TURBINE/REACTOR TRIP DUE TO LOSS OF POWER TO TURBINE EHC PUMPS WHEN MCC 2XA TRIPPED.
15-P	09/30/80	F	.0	D	5		RC	FUELXX	XENON HOLD AT 38% POWER.

SUMMARY

OCONEE 2 OPERATED ROUTINELY DURING SEPTEMBER, WITH 1 OUTAGE DUE TO ADMINISTRATIVE ERROR AND FOUR REDUCTIONS LISTED IN DETAIL ABOVE.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

OCONEE 3

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	South Carolina/Oconee 30 miles W of Greenville
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2568
5.	Design Electrical Rating (MWE Net)	887
6.	Date of Initial Criticality	September 5, 1974
7.	Date of Commercial Operation	December 16, 1974
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Keowee Lake

Utility & Contractor Information

10.	Licensee	Duke Power Company
11.	Corporate Address	422 S. Church Street Charlotte, North Carolina 28242
12.	Corporate Contact	W. O. Parker, Jr.
13.	Architect/Engineer	DPC/Bechtel
14.	Nuclear Steam Supply System	B&W
15.	Constructor	DPC

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R.C. Lewis
18.	IE Section Chief	R. MARTIN
19.	IE Resident Inspector	F. Jape (803) 882-6927
20.	Licensing Project Manager	M. B. Fairtile
21.	Docket Number	50-287
22.	License Number/Date of Issue	DPR-55, 7/19/74
23.	Public Document Room	Oconee County Library 201 S. Spring Street Walhalla, South Carolina 29691

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

REFUELING OUTAGE SCHEDULED TO BEGIN 11/80.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

ROUTINE OPERATION.

LAST IE SITE INSPECTION DATE: SEPTEMBER 23-26, 1980

INSPECTION REPORT NO: 50-287/80-26

SUMMARY

INSPECTION SUMMARY

INSPECTION JULY 1-31 (80-21): THIS ROUTINE INSPECTION INVOLVED 73 RESIDENT INSPECTOR-HOURS ON SITE IN THE AREAS OF PLANT OPERATION, SURVEILLANCE TESTING, TEST AND MEASURING EQUIPMENT, MAINTENANCE PROGRAM, PHYSICAL SECURITY, PLANT MODIFICATION TEST WITNESSING, RADIOACTIVE WASTE TRANSPORT, INSTRUCTOR LICENSING, LER FOLLOWUP, SPENT FUEL SHIPMENT, AND OFFSITE CONTAMINATION. OF THE 11 AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN 9 AREAS; 2 ITEMS OF NONCOMPLIANCE WERE FOUND IN 2 AREAS (INFRACTION - FAILURE TO MAINTAIN TRACEABILITY AND ACCOUNTABILITY OF QA MATERIAL USED ON SAFETY-RELATED SYSTEM; INFRACTION - FAILURE TO EMPLOY PROCEDURE WHEN MOVING RADIOACTIVE WASTE).

INSPECTION AUGUST 25-28 (80-22): INCLUDED REVIEW OF THE LICENSEE'S CORRECTIVE ACTIONS TO PRIOR ITEM OF NONCOMPLIANCE 79-24-01, ALSO INSPECTION OF SECURITY ORGANIZATION, BARRIERS, ACCESS CONTROLS, ALARM STATIONS, DETECTION AIDS, COMMUNICATIONS, COMPENSATORY MEASURES, POWER SUPPLY AND I&E INFORMATION NOTICE NO. 80-18. THE INSPECTION INVOLVED 9 INSPECTOR-HOURS ON SITE BY ONE NRC INSPECTOR. THE INSPECTION WAS BEGUN DURING A REGULAR PERIOD; 3 INSPECTION HOURS WERE ACCOMPLISHED DURING OFFSHIFT PERIODS. THE LICENSEE WAS FOUND TO BE IN COMPLIANCE WITH NRC REQUIREMENTS WITHIN ALL THE AREAS EXAMINED DURING THE INSPECTION. CORRECTIVE ACTION TO 79-24-01 WAS FOUND ACCEPTABLE.

INSPECTION AUGUST 1-31 (80-23): THIS ROUTINE INSPECTION INVOLVED 57 (RESIDENT) INSPECTOR-HOURS ON SITE IN THE AREAS OF PLANT OPERATIONS, SURVEILLANCE TESTING, INSTRUMENT CALIBRATION, PHYSICAL SECURITY, LER REVIEW, CONTAMINATED TENDON GALLERIES, HYDROGEN GENERATION INSIDE CONTAINMENT, AND LOAD CENTER LOCATION. OF THE EIGHT AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

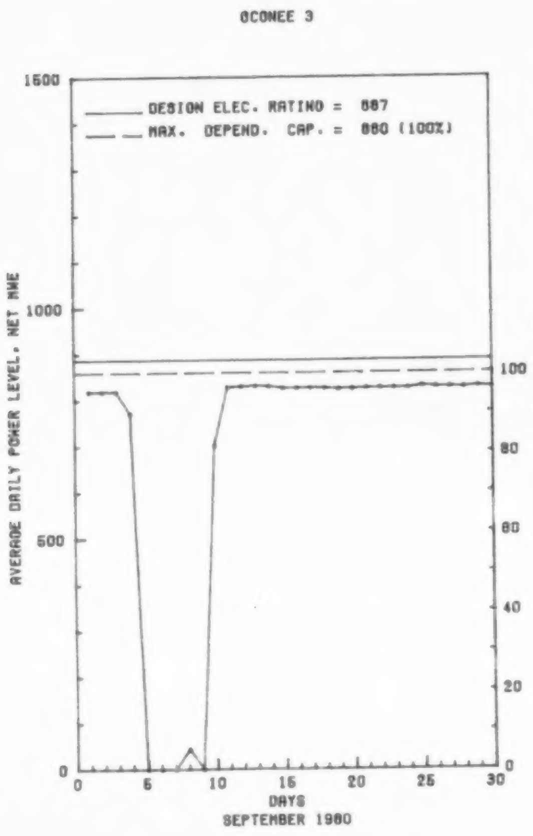
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE			

REVIEWED BY OMPA LOEB: PE Bobe **DATE:** 11/4/80

REVIEWED BY NRR DL: MB Fairtile **DATE:** 11/5/80

OCONEE 3

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-287
2. Utility Contact: J. A. Reavis (704) 373-8552
3. Licensed Thermal Power (MWt): 2568
4. Nameplate Rating (Gross MWe): 1038 x 0.9 = 934
5. Design Electrical Rating (Net MWe): 887
6. Maximum Dependable Capacity (Gross MWe): 899
7. Maximum Dependable Capacity (Net MWe): 860
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

CAPACITY FACTORS CALCULATED WITH A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	6,575.0	50,783.0
12. Hours Reactor Was Critical	616.5	4,923.1	36,817.0
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	609.5	4,833.6	35,895.5
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,517,678	12,145,366	86,516,945
17. Gross Electrical Energy (MWH)	516,480	4,186,103	29,937,364
18. Net Electrical Energy (MWH)	490,268	3,986,896	28,483,452
19. Unit Service Factor	84.7	73.5	70.7
20. Unit Availability Factor	84.7	73.5	70.7
21. Unit Capacity Factor (MDC Net)	79.2	70.5	64.8
22. Unit Capacity Factor (DER Net)	76.8	68.4	63.2
23. Unit Forced Outage Rate	15.3	13.7	17.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUELING - DECEMBER 5 - 10 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type¹	Duration (Hours)	Reason²	Method of Shutting Down Reactor³	Licensee Event Report No.	System Code⁴	Component Code⁵	Cause & Corrective Action to Prevent Recurrence
11-P	09/01/80	F	.0	A	5		ZZ	ZZZZZZ	HOLDING AT 96% POWER FOR BETTER UNIT CONTROL WITH RPS CHANNELS A AND D NOT SERVICEABLE.
6	09/05/80	F	77.7	A	1		IB	INSTRU	SHUTDOWN FOR REPAIRS TO RPS CHANNELS A AND D. COULD NOT TEST OTHER CHANNEL THAT WAS DUE.
7	09/08/80	F	32.8	G	1		CB	PUMPXX	SHUTDOWN TO CHANGE OIL IN 3A1 AND 3A2 REACTOR COOLANT PUMPS DUE TO THE ADDING OF WRONG OIL DURING PREVIOUS OUTAGE.

SUMMARY

OCONEE 3 OPERATED AT NEAR FULL POWER FOR THE MAJORITY OF THE MONTH, WITH A REDUCTION ON THE 1ST FOR EQUIPMENT FAILURE, AN OUTAGE ON THE 5TH FOR EQUIPMENT FAILURE AND AN OUTAGE ON THE 8TH FOR OPERATIONAL ERROR.

LEGEND

¹Type:
- F--Forced
- S--Scheduled

²Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

³Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

⁴Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

⁵Exhibit H: Same source.

OYSTER CREEK

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	New Jersey/Ocean 9 miles S of Toms River
2.	Electric Reliability Council	Mid-Atlantic Area Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	1930
5.	Design Electrical Rating (MWE Net)	650
6.	Date of Initial Criticality	May 3, 1969
7.	Date of Commercial Operation	December 1969
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Barnegat Bay

Utility & Contractor Information

10.	Licensee	Jersey Central Power & Light Co.
11.	Corporate Address	Madison Avenue At Punch Bowl Road Morristown, New Jersey 07960
12.	Corporate Contact	I. Finfrock, Vice President Power Generation
13.	Architect/Engineer	Burns & Roe
14.	Nuclear Steam Supply System	GE
15.	Constructor	J. A. Jones

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	IE Section Chief	R. R. KEIMIG
19.	IE Resident Inspector	L. BRIGGS (X5306) (609) 693-0702 NRC RESIDENT
20.	Licensing Project Manager	W. A. Paulson
21.	Docket Number	50-219
22.	License Number/Date of Issue	DPR-16, 4/9/69
23.	Public Document Room	Ocean County Library 15 Hooper Avenue Toms River, New Jersey 08753

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

- MANAGERIAL ITEMS:
- TECHNICAL SPECIFICATION CHANGE WAS APPROVED DELINEATING THE NEW MANAGEMENT ORGANIZATION DEVELOPED BY COMBINING MANAGEMENT AND TECHNICAL STAFF WITHIN GPU SERVICE CORPORATION, METROPOLITAN EDISON COMPANY, AND JERSEY CENTRAL POWER AND LIGHT COMPANY TO FORM THE GPU NUCLEAR GROUP.
- MEMBERS OF LICENSEE MANAGEMENT MET WITH NRC REGION I STAFF ON SEPTEMBER 25, 1980, TO DISCUSS LICENSEE PERFORMANCE. THIS MEETING WAS CONDUCTED AS PART OF THE SYSTEMATIC APPRAISAL OF LICENSEE PERFORMANCE (SALP) PROGRAM.

PLANT STATUS:

- PLANT IS OPERATING AT A NOMINAL 100 PERCENT POWER WITH AN OFFGAS RATE OF 6200 MICROCURIES PER SECOND AND A STACK RELEASE RATE OF 1060 MICROCURIES PER SECOND.

- LAST IE SITE INSPECTION DATE: 9/25/80
- INSPECTION REPORT NO: 50-219/80-31

SUMMARY

INSPECTION SUMMARY

- 50-219/80-28 - AUG 4-29: ROUTINE INSPECTION BY THE RESIDENT INSPECTORS AND ONE REGION BASED INSPECTOR (70 HRS) OF: LICENSEE ACTION ON PREVIOUS INSPECTION FINDINGS; FOLLOWUP OF OPERATIONAL EVENTS THAT OCCURRED DURING THE INSPECTION; TOURS OF THE FACILITY; LOG AND RECORD REVIEW; AND FOLLOWUP OF IE BULLETIN 80-11. THREE ITEMS OF NONCOMPLIANCE WERE IDENTIFIED: - FAILURE TO WEAR PROTECTIVE CLOTHING AS REQUIRED BY RWP; - FAILURE TO REMOVE PROTECTIVE CLOTHING BEFORE LEAVING A CONTROLLED CONTAMINATED AREA; - INADEQUATE PROCESS CONTROLS ON TEMPORARY WASTE DEMINERALIZER.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

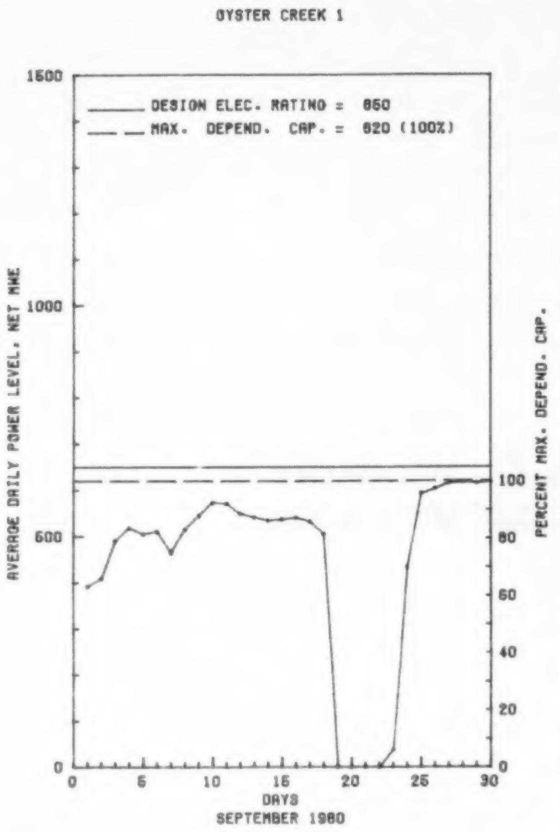
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-028/03L	07/30/80	08/28/80	TWO OF FOUR REACTOR HIGH PRESSURE SCRAM SENSORS FOUND SET ABOVE T.S. LIMITS
80-033/03L	08/06/80	08/28/80	TORUS OXYGEN CONCENTRATION WAS ABOVE 5 PERCENT. A REACTOR SHUTDOWN WAS COMMENCED THEN TERMINATED WHEN THE CONCENTRATION WAS REDUCED TO LESS THAN 5 PERCENT
80-034/03L	07/29/80	09/02/80	SBGTS NO. 1 TRIPPED DUE TO OVERLOAD DURING ROUTINE SURVEILLANCE
80-05E	08/08/80	08/26/80	LOSS OF DILUTION PUMPS TEN TIMES OVER A FOUR DAY PERIOD WHEN TEMPERATURE AT THE ROUTE 9 BRIDGE WAS ABOVE 87F
80-06E	08/11/80	08/26/80	CONDENSER CIRCULATING WATER TEMPERATURE EXCEEDED THE T.S. LIMIT OF 106 F BY .5F

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 11-4-80

REVIEWED BY NRR DL: Walt A. Paulson **DATE:** 11-5-80

OYSTER CREEK

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-219
2. Utility Contact: C. M. McClain (201) 455-8748
3. Licensed Thermal Power (MWt): 1930
4. Nameplate Rating (Gross MWe): 687.5 x 0.8 = 550
5. Design Electrical Rating (Net MWe): 650
6. Maximum Dependable Capacity (Gross MWe): 650
7. Maximum Dependable Capacity (Net MWe): 620
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): 550
10. Reasons for Restrictions, If Any:
CONDENSATE DEMINERALIZER DIFFERENTIAL PRESSURE.

ITEM 21 CALCULATED BY USING A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours In Reporting Period	720.0	6,575.0	94,439.0
12. Hours Reactor Was Critical	621.0	1,738.4	70,377.7
13. Reactor Reserve Shutdown Hours	.0	.0	468.2
14. Hours Generator On-Line	604.0	1,639.2	68,944.7
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,055,690	2,661,021	116,540,779
17. Gross Electrical Energy (MWH)	333,920	836,630	39,718,965
18. Net Electrical Energy (MWH)	319,951	794,811	38,275,824
19. Unit Service Factor	83.9	24.9	73.0
20. Unit Availability Factor	83.9	24.9	73.0
21. Unit Capacity Factor (MDC Net)	71.7	19.5	66.9
22. Unit Capacity Factor (DER Net)	68.4	18.6	62.4
23. Unit Forced Outage Rate	16.1	11.9	6.6

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
SPRING 1981
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

II

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
4	09/18/80	F	116.0	B	1		ZZ	ZZZZZZ	MAINTENANCE OUTAGE AS RESULT OF INCREASING DRYWELL LEAK RATE

SUMMARY

OYSTER CREEK OPERATED NORMALLY DURING SEPTEMBER, WITH ONE OUTAGE DUE TO AN INCREASING DRYWELL LEAK RATE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H
Same source

PALISADES

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Michigan/Van Buren 5 miles S of South Haven
2.	Electric Reliability Council	East Central Area Reliability Coordination Agreement
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2530
5.	Design Electrical Rating (MWE Net)	806
6.	Date of Initial Criticality	May 24, 1971
7.	Date of Commercial Operation	December 31, 1971
8.	Condenser Cooling Method	Forced Draft Cooling Towers
9.	Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10.	Licensee	Consumers Power Company
11.	Corporate Address	212 W. Michigan Avenue Jackson, Michigan 49201
12.	Corporate Contact	R. B. DeWitt
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Combustion Engineering
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	D.C. Boyd
19.	IE Resident Inspector	B. Jorgenson (616) 764-8971
20.	Licensing Project Manager	R. D. Silver
21.	Docket Number	50-255
22.	License Number/Date of Issue	DPR-20, 3/24/71
23.	Public Document Room	Kalamazoo Public Library 315 South Rose Street Reference Department Kalamazoo, Michigan 49006

INSPECTION STATUS

ENFORCEMENT STATUS

TECHNICAL SPECIFICATION 6.8.1 REQUIRES IMPLEMENTATION OF PROCEDURES AS RECOMMENDED IN APPENDIX A OF REGULATORY GUIDE 1.33, WHICH INCLUDES ADMINISTRATIVE PROCEDURES FOR EQUIPMENT CONTROL. LICENSEE ADMINISTRATIVE PROCEDURE 5.0, PARAGRAPH 5.1.12.7, CONTROLS RETURN OF EQUIPMENT FROM MAINTENANCE UNDER AN EQUIPMENT OUTAGE REQUEST (EOR) AND REQUIRES COMPLETION OF ADMINISTRATIVE REVIEW AND RELEASE, AND OTHER EOR SIGNOFFS, BEFORE EQUIPMENT CAN BE CONSIDERED OPERABLE. CONTRARY TO THE ABOVE, NI-003 (A WIDE-RANGE NUCLEAR POWER INSTRUMENT) WAS IMPROPERLY RETURNED FROM MAINTENANCE UNDER AN EOR ON JULY 10, 1980. THE INSTRUMENT WAS RELIED ON AS OPERABLE FOR A PLANT CRITICALITY AT 1230 HOURS (EDT), BEFORE THE EOR WAS PROPERLY COMPLETED. (8013 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENTS

- The reactor was manually tripped from significant power (88%) on August 26, 1980, following loss of a condensate pump. Plant response was normal; all systems performing as required. (80-15)

MANAGERIAL ITEMS

- A proposed imposition of a civil penalty in the amount of $16,000.00 was issued to the licensee on September 16, 1980, regarding incorrect positioning of safety-related valves on July 25-27 and August 19, 1980. (80-12)
- Effective September 1, 1980, Mr. James S. Rang assumed duties as Superintendent, Operations and Maintenance. He replaces Mr. H. W. Keiser, who left the licensee for a position at the Susquehanna Nuclear Plant. Mr. Rang formerly held the same position at the Big Rock Point Nuclear Plant. (80-15)
- Also effective September 1, 1980, the Quality Control group was transferred from the Technical Department to the Quality Assurance Department. Routine group activities and responsibilities are unchanged. (80-15)

PLANT STATUS

- Operating routinely.

LAST IE SITE INSPECTION DATE: August 4-29, 1980

INSPECTION REPORT NO: 80-15

SUMMARY

INSPECTION SUMMARY

- Inspection on July 29 and August 19-20, (80-12): Reactive, announced inspection to review the circumstance and significance associated with the reported incorrect position of containment sump isolation valve CV-3030 and misoperation of the Safety Injection and Refueling Water (SIRW) Tank isolation valve CV-3031. The inspection involved 30 inspector-hours onsite by four NRC inspectors including six inspector-hours onsite during off-shifts. Five items of noncompliance were identified (degraded ECCS pumps; degraded containment spray pumps; failure to maintain containment integrity; failure to promptly report significance of incorrect valve position and failure to follow procedures).
- Inspection on August 4-29, (80-15): Routine monthly resident inspection program activities including: operational safety; maintenance; surveillance; reportable events; IE Bulletins and Circulars; plant trip; current regulatory items; and followup on previously identified items. The inspection involved 223 inspector-hours onsite by two NRC inspectors including 38 inspector-hours onsite during off-shifts. Of the eight areas inspected, no items of noncompliance were identified in seven areas. Two items of noncompliance (plant operation with two inoperable safety-related snubbers and reporting requirements not met) were identified in the remaining one area.

NRC POWER RESTRICTIONS: 83% OF THERMAL POWER

REPORTS RECEIVED FROM LICENSEE

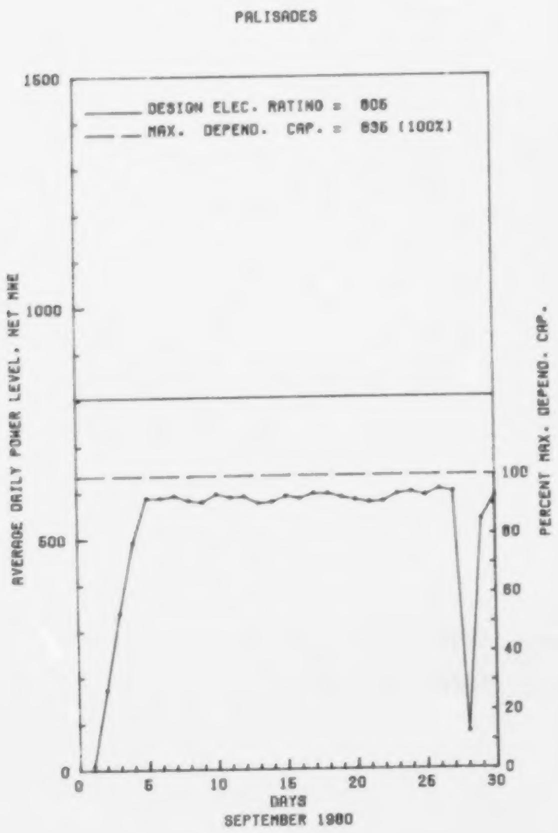
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-23/ 03L-0	07/10/80	09/02/80	Prior to startup RE 1805 and 1807 (containment high radiation detectors) were determined to be inoperable.
80-26/ 03L-0 update-09/17/80	07/19/80	08/18/80	During refilling of Safety Injection Tanks following sampling, it was observed that FT-0313 (HSPI flow to loop 2B) was inoperable.
80-27/ 01T-0	08/12/80	08/27/80	During venting of quench tank to the waste gas surge tank, an unplanned release occurred.
80-28/ 03L-0	08/07/80	09/02/80	During normal power operation, a low level alarm was received from the level switch on SI tank "A".
80-29/ 01T-0	08/19/80	09/02/80	During power operation CV-3031 was stroked closed and immediately reopened.
80-30/ 03L-0	08/11/80	09/08/80	During normal operation, the indicating lamps for CV-0437B and CV-0438B were not lit.
80-31/ 01T-0	08/26/80	09/23/80	During performance of control rod inter. testing, group 1 rods withdrew approximately 121 inches.
80-33/ 01T-0	09/11/80	09/25/80	During control rod exercising, rod 31 was momentarily misaligned, as evidenced by an 8 inch deviation alarm.
80-34/ 03L-0	08/26/80	09/26/80	With plant in hot shutdown, preparations were made to conduct check valve ISI surveillance per procedure QO-8.

REVIEWED BY OMPA LOEB: M.R. Beebe **DATE:** 11/04/80

REVIEWED BY NRR DL: T.V. Wambach **DATE:** 11/05/80

PALISADES

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-255
2. Utility Contact: P. J. Bodtke (616) 764-8913
3. Licensed Thermal Power (MWt): 2530
4. Nameplate Rating (Gross MWe): 955 x 0.85 = 812
5. Design Electrical Rating (Net MWe): 805
6. Maximum Dependable Capacity (Gross MWe): 675
7. Maximum Dependable Capacity (Net MWe): 635
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): 580
10. Reasons for Restrictions, If Any: NRC RESTRICT DECAY HEAT REMOVAL & WATER HAMMER CONCERNS.

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	6,575.0	77,006.0
12. Hours Reactor Was Critical	699.6	2,761.1	62,325.9
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	681.0	2,646.8	40,073.3
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,328,640	5,608,498	77,119,968
17. Gross Electrical Energy (MWH)	406,620	1,723,410	23,755,650
18. Net Electrical Energy (MWH)	378,679	1,616,593	22,287,368
19. Unit Service Factor	94.6	40.3	52.0
20. Unit Availability Factor	94.6	40.3	52.0
21. Unit Capacity Factor (MDC Net)	82.8	38.7	45.6
22. Unit Capacity Factor (DER Net)	65.3	30.5	36.0
23. Unit Forced Outage Rate	5.4	12.7	35.0

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): FIRE PROTECTION MODS, NOVEMBER 1 - 2 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
17	08/26/80	F	22.7	A	4				CONDENSATE PUMP TRIP.
18	09/28/80	F	16.8	A	1				SHORT ON POWER SUPPLY TO TURBINE CIRCUITRY.

SUMMARY

PALISADES OPERATED UNDER RESTRICTION FOR SEPTEMBER, WITH CONCLUSION OF AN OUTAGE ON THE 1ST FOR CONDENSATE PUMP TRIP AND ANOTHER OUTAGE ON THE 28TH FOR A SHORT CIRCUIT TO THE TURBINE CIRCUITRY POWER SUPPLY.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

PEACH BOTTOM 2

FACILITY DATA

Facility Description

1. Location (state/county) (nearest population center)	Pennsylvania/York 19 miles S of Lancaster
2. Electric Reliability Council	Mid-Atlantic Area Council
3. Type of Reactor	BWR
4. Power Level (MWT)	3293
5. Design Electrical Rating (MWE Net)	1065
6. Date of Initial Criticality	September 16, 1973
7. Date of Commercial Operation	July 5, 1974
8. Condenser Cooling Method	Once Through with Cooling Towers
9. Condenser Cooling Water Source	Conowingo Pond/Susquehanna River

Utility & Contractor Information

10. Licensee	Philadelphia Electric Company
11. Corporate Address	2301 Market Street Philadelphia, Pennsylvania 19101
12. Corporate Contact	S. L. Daltroff, Vice President Electric Production
13. Architect/Engineer	Bechtel
14. Nuclear Steam Supply System	GE
15. Constructor	Bechtel

Regulatory Information

16. IE Region responsible	I
17. IE Branch Chief	E. Brunner
18. IE Section Chief	E. McCabe
19. IE Resident Inspector	C. Cowgill 717-456-7614
20. Licensing Project Manager	D. Verrelli
21. Docket Number	50-277
22. License Number/Date of Issue	DPR-44, 8/8/73
23. Public Document Room	Government Publications Section State Library of Pennsylvania Education Building Commonwealth and Walnut Street Harrisburg, Pennsylvania 17126

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

+ FACILITY ITEMS (PLANS AND PROCEDURES):

+ THE STATION OPERATIONS ENGINEER, MR. C. E. ANDERSON, WAS TRANSFERRED TO THE SUSQUEHANNA TEST BRANCH; MR. S. R. ROBERTS WAS NAMED AS HIS REPLACEMENT.

+ AN NRC RESEARCH TEAM WAS ON SITE SEPTEMBER 2, 1980 TO GATHER INFORMATION REGARDING ESF STATUS MONITORING IN THE CONTROL ROOM.

+ MANAGERIAL ITEMS:

+ THE OFFSITE SAFETY REVIEW COMMITTEE MET ON SITE ON SEPTEMBER 12, 1980.

PLANT STATUS:

+ THE UNIT WAS OPERATED AT NOMINALLY FULL POWER WITH OFFGAS RATE OF 3246 MICROCURIES/SECOND AND A COMBINED UNIT 2 AND 3 STACK GAS RELEASE RATE OF ABOUT 270 MICROCURIES PER SECOND.

+ LAST IE SITE INSPECTION DATE: 9/1-30/80

+ INSPECTION REPORT NO: 50-277/80-29

SUMMARY

INSPECTION SUMMARY

+ 50-277/80-03 - FEB 1-29: ROUTINE, ONSITE REGULAR AND OFF SHIFT INSPECTIONS BY THE RESIDENT INSPECTORS (12 HRS). AREAS EXAMINED INCLUDED OPERATIONAL SAFETY VERIFICATIONS, PHYSICAL SECURITY, FOLLOW-UP ON PRIOR IDENTIFIED ITEMS, REVIEW OF PERIODIC AND SPECIAL REPORTS, CONTROL ROOM INSPECTIONS, LER REVIEW ON SITE AND IN OFFICE, AND RADIATION PROTECTION. TWO NONCOMPLIANCES WERE IDENTIFIED: FAILURE TO MAINTAIN ACCESS CONTROLS; - FAILURE TO USE TEMPORARY MONITORS OR INITIATE AN ORDERLY SHUTDOWN WITH THE OFF-GAS MONITORING SYSTEM OUT OF SERVICE.

+ 50-277/80-05 - MAR 1-31: ROUTINE, ONSITE REGULAR AND BACKSHIFT INSPECTION BY THE RESIDENT INSPECTORS (49 HRS). AREAS INSPECTED INCLUDED PREPARATIONS FOR A REFUELING OUTAGE, ACCESSIBLE PORTIONS OF THE FACILITIES, RADIATION PROTECTION, MAINTENANCE ACTIVITIES, PHYSICAL SECURITY, PLANT OPERATIONS, FACILITY TOURS, CONTROL ROOM INSPECTION, FOLLOWUP ON PREVIOUSLY IDENTIFIED ITEMS, IMPLEMENTATION OF THREE MILE ISLAND LESSONS LEARNED REQUIREMENTS, BULLETIN AND INFORMATION NOTICE FOLLOWUP, AND REVIEW OF PERIODIC AND SPECIAL REPORTS. FOUR NONCOMPLIANCES WERE IDENTIFIED: (1) FAILURE TO FOLLOW FUEL RECEIPT INSPECTION PROCEDURES; (2) FAILURE TO WEAR PERSONNEL DOSIMETRY; (3) FAILURE TO ADHERE TO ESCORT REQUIREMENTS; (4) FAILURE TO ESTABLISH WRITTEN PROCEDURES.

+ 50-277/80-11 - APR 22-MAY 19: ROUTINE, UNANNOUNCED INSPECTION BY 3 REGION-BASED INSPECTORS (80 HRS) OF THE RADIATION PROTECTION PROGRAM FOR OPERATION AND REFUELING, INCLUDING: PERSONNEL QUALIFICATIONS; LICENSEE AUDIT; TRAINING; RADIATION PROTECTION PROCEDURES; ADVANCE PLANNING AND PREPARATION FOR REFUELING; INSTRUMENTS AND EQUIPMENT; EXPOSURE CONTROL; POSTING; LABELING, AND CONTROL; NOTIFICATIONS AND REPORTS; SURVEYS; FOLLOWUP ON PREVIOUS ENFORCEMENT ACTION, AND LER REVIEW. THREE NONCOMPLIANCES WERE IDENTIFIED: FAILURE TO ADHERE TO RADIATION PROTECTION PROCEDURES; FAILURE TO MAINTAIN CURRENT PROCEDURES AT THE WORK LOCATION; FAILURE TO MAINTAIN POSTING OF FORM NRC-3 CURRENT IN ACCORDANCE WITH 10 CFR 19.11.

+ 50-277/80-14 - MAY 1-31: ROUTINE ONSITE AND BACKSHIFT INSPECTION BY THE RESIDENT INSPECTORS (27 HRS). AREAS INSPECTED INCLUDED PLANT OPERATIONS (OPERATIONAL SAFETY VERIFICATION), FACILITY TOURS, CONTROL ROOM INSPECTIONS, NONROUTINE EVENT REVIEW, IE BULLETIN AND CIRCULAR FOLLOW-UP, RADIATION PROTECTION, PHYSICAL SECURITY, AND REVIEW OF PERIODIC REPORTS. ONE NONCOMPLIANCE WAS IDENTIFIED. FAILURE TO MAINTAIN TWO FIRE STATIONS UNOBSTRUCTED FOR ACCESSIBILITY.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE			

+ 50-277/80-16 - JUN 1-30: ROUTINE, ONSITE REGULAR AND BACKSHIFT INSPECTIONS BY A RESIDENT INSPECTOR (17 HRS). AREAS INSPECTED INCLUDED PLANT OPERATIONS, FACILITY TOURS, CONTROL ROOM INSPECTIONS, REVIEW OF PERIODIC REPORTS, HOUSEKEEPING, RADIATION PROTECTION, REACTOR CHEMISTRY, LER REVIEW ONSITE AND FOLLOWUP ON PRIOR IDENTIFIED ITEMS. ONE NONCOMPLIANCE WAS IDENTIFIED: FAILURE TO PROPERLY LOG VALVE POSITIONS AS REQUIRED WHEN AN ISOLATION VALVE IS INOPERABLE.

+ 50-277/80-19 - JUN 11: SPECIAL, UNANNOUNCED INSPECTION BY TWO REGION-BASED INSPECTORS (12 HRS) TO REVIEW THE LICENSEE'S EVALUATION OF AN INDIVIDUAL'S EXPOSURE TO RADIATION, INCLUDING RADIATION PROTECTION PROCEDURES; SURVEYS; PERSONNEL TRAINING; EXPOSURE CONTROL; AREA POSTING AND CONTROL; AND DOSIMETRY PROGRAM. NO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED.

+ 50-277/80-25 - JUL 9-11 AND 24: ROUTINE, UNANNOUNCED INSPECTION BY A REGION-BASED INSPECTOR (31 HRS) OF ACTIVITIES ASSOCIATED WITH CORE SPRAY MODIFICATION/CAPPING OF CONTROL ROD DRIVE (CRD) HYDRAULIC RETURN LINE AND DOCUMENTATION OF REVIEWS OF INSERVICE INSPECTION (ISI) ACTIVITIES. NO NONCOMPLIANCES WERE IDENTIFIED.

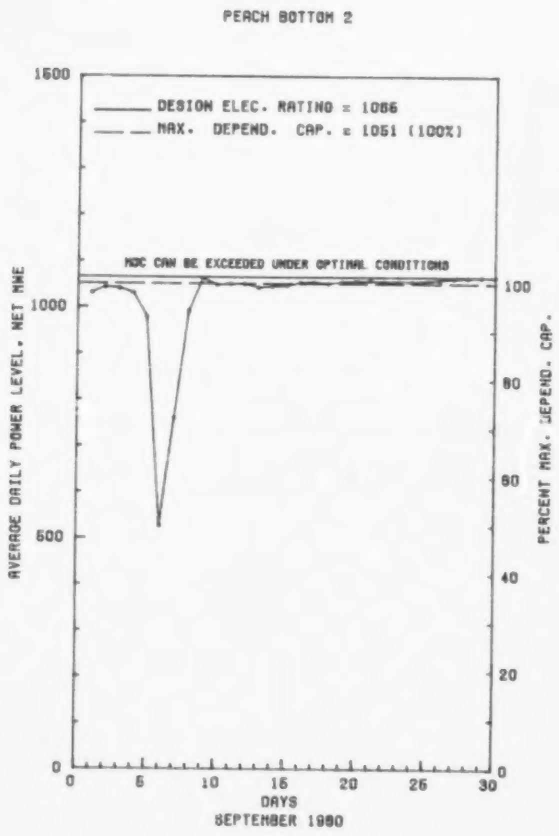
+ 50-277/80-26 - JUL 24-28: ROUTINE, UNANNOUNCED INSPECTION BY TWO REGION-BASED INSPECTORS (53 HRS) OF: CONTAINMENT INTEGRATED LEAK RATE TEST ACTIVITIES; CONTAINMENT LOCAL LEAK RATE TEST PROCEDURES; SHUTDOWN PLANT ACTIVITIES; AND LICENSEE ACTION ON PREVIOUS INSPECTION FINDINGS. TWO NONCOMPLIANCES WERE IDENTIFIED: FAILURE TO PROPERLY POST A RADIOLOGICALLY CONTROLLED AREA; FAILURE TO ISSUE PROPER DOSIMETRY DEVICES.

REVIEWED BY OMPA LOEB: J. J. Rizzo **DATE:** 11-4-80

REVIEWED BY NRR DL: **DATE:** 11-6-80

PEACH BOTTOM 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-277
2. Utility Contact: W. M. Alden (215) 841-5022
3. Licensed Thermal Power (MWt): 3293
4. Nameplate Rating (Gross MWe): 1280 x 0.9 = 1152
5. Design Electrical Rating (Net MWe): 1065
6. Maximum Dependable Capacity (Gross MWe): 1098
7. Maximum Dependable Capacity (Net MWe): 1051
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	6,575.0	54,719.0
12. Hours Reactor Was Critical	720.0	2,679.0	40,134.0
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	720.0	2,461.3	39,444.1
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	2,275,675	7,076,193	113,841,837
17. Gross Electrical Energy (MWH)	761,640	2,337,020	37,395,250
18. Net Electrical Energy (MWH)	734,477	2,209,684	35,825,469
19. Unit Service Factor	100.0	37.4	72.1
20. Unit Availability Factor	100.0	37.4	72.1
21. Unit Capacity Factor (MDC Net)	97.1	32.0	62.3
22. Unit Capacity Factor (DER Net)	95.8	31.6	61.5
23. Unit Forced Outage Rate	.0	2.2	6.1

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
11	09/05/80	S	.0	H	5		RC	ZZZZZZ	LOAD DROP FOR CONTROL ROD PATTERN ADJUSTMENT.

SUMMARY

PEACH BOTTOM 2 OPERATED AT OR NEAR FULL POWER FOR SEPTEMBER, WITH EXCEPTION OF ONE REDUCTION ON THE 5TH FOR A CONTROL ROD PATTERN ADJUSTMENT.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

PEACH BOTTOM 3

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Pennsylvania/York 19 miles S of Lancaster
2.	Electric Reliability Council	Mid-Atlantic Area Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	3293
5.	Design Electrical Rating (MWE Net)	1065
6.	Date of Initial Criticality	August 7, 1974
7.	Date of Commercial Operation	December 23, 1974
8.	Condenser Cooling Method	Once Through with Cooling Towers
9.	Condenser Cooling Water Source	Conowingo Pond/Susquehanna River

Utility & Contractor Information

10.	Licensee	Philadelphia Electric Company
11.	Corporate Address	2301 Market Street Philadelphia, Pennsylvania 19101
12.	Corporate Contact	S. L. Daltroff, Vice President Electric Production
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	GE
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	IE Section Chief	E. McCabe
19.	IE Resident Inspector	C. Cowgill (717-456-7614)
20.	Licensing Project Manager	D. Verrelli
21.	Docket Number	50-278
22.	License Number/Date of Issue	DPR-56, 7/2/74
23.	Public Document Room	Government Publications Section State Library of Pennsylvania Education Building Commonwealth and Walnut Street Harrisburg, Pennsylvania 17126

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

+ FACILITY ITEMS (PLANS AND PROCEDURES):

+ THE STATION OPERATIONS ENGINEER, MR. C. E. ANDERSON, WAS TRANSFERRED TO THE SUSQUEHANNA TEST BRANCH; MR. S. R. ROBERTS WAS NAMED AS HIS REPLACEMENT.

+ AN NRC RESEARCH TEAM WAS ON SITE, SEPTEMBER 2, 1980, TO GATHER INFORMATION REGARDING ESF STATUS MONITORING IN THE CONTROL ROOM.

MANAGERIAL ITEMS:

+ THE OFFSITE SAFETY REVIEW COMMITTEE MET ON SITE ON SEPTEMBER 12, 1980.

PLANT STATUS:

+ THE UNIT WAS OPERATED AT NOMINALLY FULL POWER WITH AN OFFGAS RATE OF 3496 MICROCURIES/SECOND AND A COMBINED UNIT 2 AND 3 STACK GAS RELEASE RATE OF ABOUT 270 MICROCURIES/SECOND.

+ LAST IE SITE INSPECTION DATE: 9/1-30/80

+ INSPECTION REPORT NO: 50-278/80-21

SUMMARY

INSPECTION SUMMARY

+ 50-278/80-03 - FEB 1-29: ROUTINE, ONSITE REGULAR AND OFFSHIFT INSPECTIONS BY THE RESIDENT INSPECTORS (12 HRS). AREAS EXAMINED INCLUDED OPERATIONAL SAFETY VERIFICATIONS, PHYSICAL SECURITY, FOLLOW-UP ON PRIOR IDENTIFIED ITEMS, REVIEW OF PERIODIC AND SPECIAL REPORTS, CONTROL ROOM INSPECTIONS, LER REVIEW ON SITE AND IN-OFFICE AND RADIATION PROTECTION. ONE NONCOMPLIANCE WAS IDENTIFIED: FAILURE TO MAINTAIN ACCESS CONTROLS.

+ 50-278/80-05 - MAR 1-31: ROUTINE, ONSITE REGULAR AND BACKSHIFT INSPECTION BY THE RESIDENT INSPECTORS (44 HRS). AREAS INSPECTED INCLUDED PREPARATIONS FOR A REFUELING OUTAGE, ACCESSIBLE PORTIONS OF THE FACILITY; RADIATION PROTECTION, MAINTENANCE ACTIVITIES, PHYSICAL SECURITY, PLANT OPERATIONS, FACILITY TOURS, CONTROL ROOM INSPECTION, FOLLOWUP ON PREVIOUSLY ITEMS, IMPLEMENTATION OF THREE MILE ISLAND LESSONS LEARNED REQUIREMENTS, BULLETINS AND INFORMATION NOTICE FOLLOWUP, AND REVIEW OF PERIOD AND SPECIAL REPORTS. FOUR NONCOMPLIANCES WERE IDENTIFIED: (1) FAILURE TO FOLLOW FUEL RECEIPT INSPECTION PROCEDURES; (2) FAILURE TO WEAR PERSONNEL DOSIMETRY; (3) FAILURE TO ADHERE TO ESCORT REQUIREMENTS (4) FAILURE TO ESTABLISH WRITTEN PROCEDURES.

+ 50-278/80-11 - APR 22 - MAY 19: ROUTINE, UNANNOUNCED INSPECTION BY 3 REGION-BASED INSPECTORS (80 HRS; OF THE RADIATION PROTECTION PROGRAM FOR OPERATION AND REFUELING, INCLUDING: PERSONNEL QUALIFICATION; LICENSEE AUDIT; TRAINING; RADIATION PROTECTION PROCEDURES; ADVANCE PLANNING AND PREPARATION FOR REFUELING; INSTRUMENTS AND EQUIPMENT; EXPOSURE CONTROL; POSTING, LABELING AND CONTROL; NOTIFICATIONS AND REPORTS; SURVEYS; FOLLOWUP ON PREVIOUS ENFORCEMENT ACTION, AND LER REVIEW. TWO NONCOMPLIANCES WERE IDENTIFIED: (1) FAILURE TO MAINTAIN CURRENT PROCEDURES AT THE WORK LOCATION; (2) FAILURE TO MAINTAIN POSTING OF FORM NRC-3 CURRENT IN ACCORDANCE WITH 10 CFR 19.11.

+ 50-278/80-12 - MAY 1-31: ROUTINE ONSITE REGULAR AND BACKSHIFT INSPECTION BY THE RESIDENT INSPECTORS (26 HRS). AREAS INSPECTED INCLUDED PLANT OPERATIONS (OPERATIONAL SAFETY VERIFICATION), FACILITY TOURS, CONTROL ROOM INSPECTIONS, NON-ROUTINE EVENT REVIEW, IE BULLETIN AND CIRCULAR FOLLOW-UP, RADIATION PROTECTION, PHYSICAL SECURITY, AND REVIEW OF PERIODIC REPORTS. NO NONCOMPLIANCES WERE IDENTIFIED.

+ 50-278/80-14 - JUN 1-30: ROUTINE, ONSITE REGULAR AND BACKSHIFT INSPECTIONS BY A RESIDENT INSPECTOR (17 HRS). AREAS INSPECTED INCLUDED PLANT OPERATIONS, FACILITY TOURS, CONTROL ROOM INSPECTIONS, REVIEW OF PERIODIC REPORTS, HOUSEKEEPING, RADIATION PROTECTION, REACTOR CHEMISTRY, LER REVIEW ONSITE AND FOLLOWUP ON PRIOR ITEMS. ONE NONCOMPLIANCE WAS IDENTIFIED: FAILURE TO PROPERLY LOG VALVE POSITIONS AS REQUIRED WHEN AN ISOLATION VALVE IS INOPERABLE.

+ 50-278/80-19 - JUL 24-28: ROUTINE, UNANNOUNCED INSPECTION BY TWO REGION-BASED INSPECTORS (1 HR), OF LICENSEE ACTION ON PREVIOUS INSPECTION FINDINGS. NO NONCOMPLIANCES WERE IDENTIFIED.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

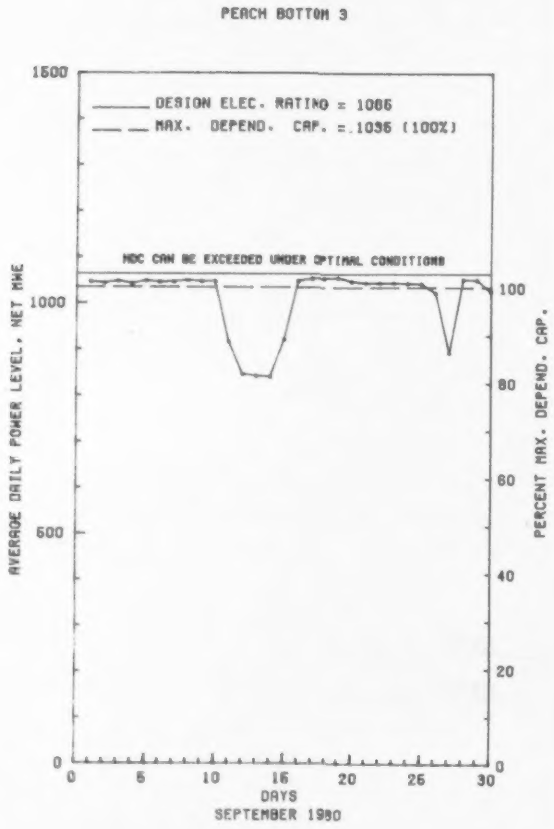
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE			

REVIEWED BY OMPA LOEB: John J. Rizzo DATE: 11-4-80

REVIEWED BY NRR DL: DATE: 11-6-80

PEACH BOTTOM 3

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-278
2. Utility Contact: W. M. Alden (215) 841-5022
3. Licensed Thermal Power (MWt): 3293
4. Nameplate Rating (Gross MWe): 1280 x 0.9 = 1152
5. Design Electrical Rating (Net MWe): 1065
6. Maximum Dependable Capacity (Gross MWe): 1098
7. Maximum Dependable Capacity (Net MWe): 1035
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	6,575.0	50,61[illegible].0
12. Hours Reactor Was Critical	720.0	5,428.0	40,274.3
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	720.0	5,258.2	39,190.0
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	2,295,019	16,726,292	110,945,446
17. Gross Electrical Energy (MWH)	755,010	5,558,260	36,144,120
18. Net Electrical Energy (MWH)	728,822	5,353,859	34,698,659
19. Unit Service Factor	100.0	80.0	77.4
20. Unit Availability Factor	100.0	80.0	77.4
21. Unit Capacity Factor (MDC Net)	97.8	78.7	66.2
22. Unit Capacity Factor (DER Net)	95.0	76.5	64.4
23. Unit Forced Outage Rate	.0	12.5	7.7

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	09/11/80	F	.0	A	5				REPAIR OF "B" CONDENSATE PUMP SHAFT SLEEVE AND THROTTLE BRUSHING.
	09/27/80	F	.0	A	5				CONDENSER LEAK INVESTIGATIONS (2).

SUMMARY

PEACH BOTTOM 3 OPERATED AT NEAR FULL POWER FOR SEPTEMBER REPORTING PERIOD, WITH REDUCTIONS FOR CONDENSATE PUMP REPAIRS AND INVESTIGATION OF CONDENSER LEAKS.

LEGEND

[1] Type:
- F–Forced
- S–Scheduled

[2] Reason:
- A–Equipment Failure
- B–Maintenance or Test
- C–Refueling
- D–Regulatory Restriction
- E–Operator Training & License Examination
- F–Administrative
- G–Operational Error
- H–Other

[3] Method:
- 1–Manual
- 2–Manual Scram
- 3–Automatic Scram
- 4–Continuations
- 5–Load Reduction
- 9–Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

PILGRIM

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Massachusetts/Plymouth 4 miles SE of Plymouth
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	BWR
4.	Licensee Power Level (MWT)	1998
5.	Design Electrical Rating (MWE Net)	655
6.	Date of Initial Criticality	June 16, 1972
7.	Date of Commercial Operation	December 1972
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Cape Cod Bay

Utility & Contractor Information

10.	Licensee	Boston Edison Company
11.	Corporate Address	800 Boylston Street Boston, Massachusetts 02199
12.	Corporate Contact	A. Corey, Vice President
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	GE
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	IE Section Chief	T. MARTIN
19.	IE Resident Inspector	J. JOHNSON
20.	Licensing Project Manager	M. WILLIAMS x27872
21.	Docket Number	50-293
22.	License Number/Date of Issue	DPR-35, 6/8/72
23.	Public Document Room	Plymouth Public Library North Street Plymouth, Massachusetts 02360

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

```
   INSPECTION  STATUS

   OTHER ITEMS OF INTEREST

+  SYSTEMS AND COMPONENT PROBLEMS:

+  1.  HPCI EXHAUST LINE SNUBBER/WATER HAMMER PROBLEMS ARE UNDER
       INVESTIGATION

+  2.  DRYWELL AMBIENT AND REACTOR PRESSURE VESSEL LEG TEMERA-
       TURES WERE OBSERVED TO BE HIGHER THAN DESIGN

+  3.  FOLLOWING THE RETURN TO SERVICE OF A CONDENSATE DEMINERALIZER,
       THE PLANT EXPERIENCED A MAIN STEAM HIGH RADIATION SIGNAL,
       MSIV CLOSURE, AND SCRAM ON 10/1/80. A SAFETY RELIEF VALVE
       STUCK OPEN AFTER BEING MANUALLY OPERATED TO ASSIST IN PRESSURE
       CONTROL.

   PLANT STATUS:

+  THE PLANT IS CURRENTLY IN COLD SHUTDOWN PENDING INVESTIGATION
   OF THE SCRAM OF 10/1/80.

+ LAST IE SITE INSPECTION DATE:  9/1-30/80

+        INSPECTION REPORT NO:  50-293/80-27

   SUMMARY

   INSPECTION SUMMARY

+  NONE
```

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-038/ 01T	08/18/80	08/29/80	MODE SWITCH IN "SHUTDOWN" SCRAM WAS INOPERABLE IN RUN MODE DUE TO DESIGN OF RPS LOGIC AND LOSS AND REGAIN OF RPS POWER SUPPLY
80-039/ 01T	08/02/80	08/29/80	APRM FLUX SCRAM SETTINGS NON-CONSERVATIVE WITH TECH SPECS
80-040/ 03L	07/24/80	08/22/80	'A' STANDBY GAS TREATMENT FAN BREAKER FOUND OPEN AFTER AREA CLEANUP
80-041/ 03L	07/24/80	08/22/80	OXYGEN ANALYZER VALVES FAILED CLOSING TIMES
80-042/ 04T	08/20/80	08/26/80	ANOMOLOUS MEASUREMENT. COBALT-60 IN MUSSELS FROM DISCHARGE CANAL
80-043/ 04L	08/01/80	08/28/80	HIGH REACTOR WATER CONDUCTIVITY DUE TO 'D' CONDENSATE DEMIN RESIN BREAKTHROUGH
80-044/ 03L	08/01/80	08/29/80	RCIC INBOARD ISOL. VALVE MOTOR BURNED OUT
80-045/ 03L	08/04/80	09/02/80	RCIC OUTBOARD ISOL. VALVE WOULD NOT CLOSE
80-046/ 03L	08/02/80	08/29/80	YARWAY LEVEL SWITCH SETPOINT NJT CORRECT
80-047/ 03L	08/01/80	09/02/80	'D' SRV FAILED TO OPEN ON DEMAND
80-048/ 03L	08/07/80	09/05/80	HPCI RCIC FIRE ALARM INOPERABLE IN CONTROL ROOM
80-049/ 03L	08/10/80	09/09/80	SALT SERVICE WATER PUMP TRIP DURING PERIODS OF HIGH INJECTION TEMPERATURE
80-051/ 03L	08/13/80	09/11/80	SETPOINY DRIFT - RV LEVEL SWITCH (L1S 263-72B)
80-052/ 03L	08/12/80	09/11/80	CONTAINMENT ATMOSPHERIC MONITOR INOP
80-053/ 03L	08/14/80	09/12/80	HPCI EXHAUST LINE SNUBBER FAILURES

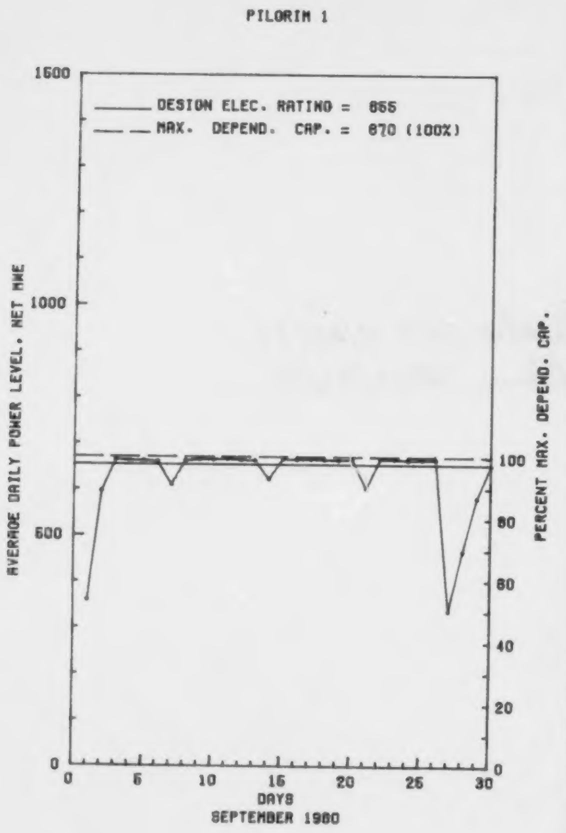
REVIEWED BY OMPA LOEB: ____________ **DATE:** 11-4-80

REVIEWED BY NRR DL: ____________ **DATE:** 11/5/80

best copy available

PILGRIM

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-293
2. Utility Contact: G. G. Whitney (617) 746-7900
3. Licensed Thermal Power (MWt): 1998
4. Nameplate Rating (Gross MWe): 780 x 0.87 = 678
5. Design Electrical Rating (Net MWe): 655
6. Maximum Dependable Capacity (Gross MWe): 690
7. Maximum Dependable Capacity (Net MWe): 670
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	6,575.0	68,471.0
12. Hours Reactor Was Critical	720.0	3,256.8	48,237.7
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	716.7	3,113.8	46,666.3
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,356,384	5,616,936	78,708,048
17. Gross Electrical Energy (MWH)	468,280	1,917,570	26,082,404
18. Net Electrical Energy (MWH)	450,604	1,844,023	25,050,146
19. Unit Service Factor	99.5	47.4	68.2
20. Unit Availability Factor	99.5	47.4	68.2
21. Unit Capacity Factor (MDC Net)	93.4	41.9	54.6
22. Unit Capacity Factor (DER Net)	95.5	42.8	55.9
23. Unit Forced Outage Rate	.0	3.1	10.1

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
7	08/30/80	S	3.3	A	4		CH	HTEXCH	SCHEDULED PLANT OUTAGE FOR 4TH POINT HEATER TUBE LEAK REPAIR
8	09/27/80	S	.0	B	5		WB	HTEXCH	RBCCW HEAT EXCHANGER OUT FOR CLEANING ON SALT WATER SIDE.

SUMMARY

PILGRIM RETURNED ON-LINE ON THE 1ST FROM A REPAIR OUTAGE AND OPERATED NORMALLY FOR THE MONTH WITH ONE REDUCTION FOR CLEANING HEAT EXCHANGER.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

POINT BEACH 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Wisconsin/Manitowoc 15 miles N of Manitowoc
2.	Electric Reliability Council	Mid-America Interpool Network
3.	Type of Reactor	PWR
4.	Power Level (MWT)	1518
5.	Design Electrical Rating (MWE Net)	497
6.	Date of Initial Criticality	November 2, 1970
7.	Date of Commercial Operation	December 21, 1970
8.	Condenser Cooling Method	Straight Through
9.	Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10.	Licensee	Wisconsin Electric Power Co.
11.	Corporate Address	231 West Michigan Milwaukee, Wisconsin
12.	Corporate Contact	S. Burstein
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	R. F. Warnick
19.	IE Resident Inspector	W. Guldemond
20.	Licensing Project Manager	C. M. Trammell
21.	Docket Number	50-266
22.	License Number/Date of Issue	DPR-24 issued 10/05/70
23.	Public Document Room	Joseph Mann Public Library Two Rivers, Wisconsin Manitowoc Public Library Manitowoc, Wisconsin (Temporary LPDR for spent fuel pool capacity increase amendment only

INSPECTION STATUS

ENFORCEMENT STATUS

TECHNICAL SPECIFICATIONS 15.6.8.1 STATES IN PART THAT THE PLANT SHALL BE OPERATED AND MAINTAINED IN ACCORDANCE WITH APPROVED PROCEDURES. RADIOLOGICAL CONTROL PROCEDURES ARE PROMULGATED THROUGH THE POINT BEACH NUCLEAR PLANT HEALTH PHYSICS ADMINISTRATIVE CONTROL POLICIES AND PROCEDURES MANUAL. (A) PART 2.1 OF THE MANUAL STATES IN PART THAT NORMAL ENTRY TO AND EXIT FROM THE CONTROLLED ZONE SHALL BE THROUGH THE TWO DESIGNATED ACCESS CONTROL POINTS ONLY. PART 2.5 STATES IN PART THAT PROTECTIVE CLOTHING REQUIREMENTS WITHIN CONTROLLED AREAS ARE ESTABLISHED BY THE CHEMISTRY AND HEALTH PHYSICS GROUP. CONTRARY TO THE ABOVE, ON JULY 1, 1980, A LICENSEE EMPLOYEE WAS OBSERVED ENTERING THE CONTROLLED ZONE AT THE SOUTH SIDE OF THE FACADE VIA A NONDESIGNATED ACCESS CONTROL POINT, NOT WEARING SPECIFIED PROTECTIVE CLOTHING, AND REMOVING MATERIAL FROM THE CONTROLLED AREA; AND (B) PART 2.4 OF THE MANUAL STATES IN PART THAT ALL RADIOACTIVE MATERIAL OUTSIDE CONTROLLED AREAS ARE TO BE TAGGED WITH A PROPERLY EXECUTED RADIOACTIVE MATERIAL TAG AND EITHER PACKAGED OR SEALED TO PREVENT THE RELEASE OF CONTAMINATION. CONTRARY TO THE ABOVE, ON JULY 2, 1980, AN OPEN TANK LABELLED "RADIOACTIVE MATERIAL" AND CONTAINING LOW LEVEL CONTAMINATION WAS LOCATED IN THE TURBINE BUILDING OUTSIDE OF A CONTROLLED AREA WITHOUT HAVING A PROPERLY EXECUTED RADIOACTIVE MATERIAL TAG.
(8012 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

- The unit operated routinely during the month.

LAST IE SITE INSPECTION DATE: July 17-22, 1980

INSPECTION REPORT NO: 80-13

SUMMARY

INSPECTION SUMMARY

- No inspections conducted.

REPORTS RECEIVED FROM LICENSEE

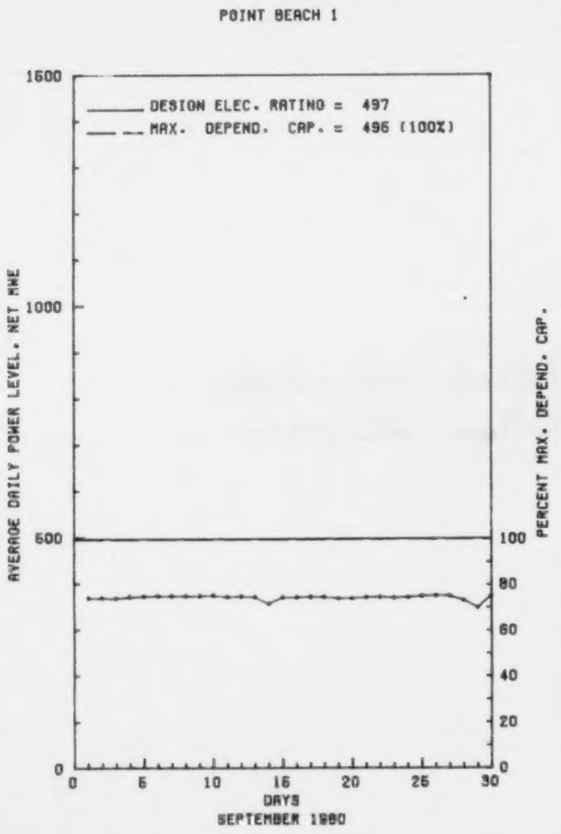
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-09/01T-0	07/28/80	08/11/80	Two dripping explosive plugs in "A" SG and two wet end plugs and "B" SG one dripping tube and one wet end plug.
			Annual results and data report for 1978. (Special Report)
80-010/01L-0	08/27/80	09/18/80	Stem cleaning and packing adjustment of containment spray pump recirculation phase suction valves. Operability testing not done prior to adjusting 2nd redundant valve.

REVIEWED BY OMPA LOEB: MR Beebe DATE: 11/05/80

REVIEWED BY NRR DL: DATE: 11/06/80

POINT BEACH 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-266
2. Utility Contact: C.W. FAY (414) 277-2811
3. Licensed Thermal Power (MWt): 1518
4. Nameplate Rating (Gross MWe): 582 x 0.9 = 524
5. Design Electrical Rating (Net MWe): 497
6. Maximum Dependable Capacity (Gross MWe): 519
7. Maximum Dependable Capacity (Net MWe): 495
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): 390
10. Reasons for Restrictions, If Any:
INHIBIT CORROSION OF STEAM GENERATOR TUBES.

ITEM 21 CALCULATED BY USING A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	6,575.0	86,807.0
12. Hours Reactor Was Critical	720.0	5,480.8	71,997.3
13. Reactor Reserve Shutdown Hours	.0	19.5	601.7
14. Hours Generator On-Line	720.0	5,330.2	69,724.[illegible]
15. Unit Reserve Shutdown Hours	.0	152.8	755.2
16. Gross Thermal Energy (MWH)	871,153	6,429,219	97,182,014
17. Gross Electrical Energy (MWH)	281,580	2,088,800	32,722,550
18. Net Electrical Energy (MWH)	266,055	1,972,570	31,162,748
19. Unit Service Factor	100.0	81.1	80.3
20. Unit Availability Factor	100.0	83.4	81.2
21. Unit Capacity Factor (MDC Net)	74.7	60.6	73.7
22. Unit Capacity Factor (DER Net)	74.4	60.4	72.2
23. Unit Forced Outage Rate	.0	.1	3.4

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
REFUELING - 11/14/80 - APPROX. 5 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	NONE								

SUMMARY

POINT BEACH 1 OPERATED UNDER A UTILITY IMPOSED RESTRICTION FOR THE MONTH OF SEPTEMBER.

LEGEND

[1] Type:
F-Forced
S-Scheduled

[2] Reason:
A-Equipment Failure
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error
H-Other

[3] Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Continuations
5-Load Reduction
9-Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

POINT BEACH 2

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Wisconsin/Manitowoc 15 miles N of Manitowoc
2.	Electric Reliability Council	Mid-America Interpool Network
3.	Type of Reactor	PWR
4.	Power Level (MWT)	1518
5.	Design Electrical Rating (MWE Net)	497
6.	Date of Initial Criticality	May 30, 1972
7.	Date of Commercial Operation	October 1, 1972
8.	Condenser Cooling Method	Straight Through
9.	Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10.	Licensee	Wisconsin Electric Power Co.
11.	Corporate Address	231 West Michigan Milwaukee, Wisconsin
12.	Corporate Contact	S. Burstein
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	R. F. Warnick
19.	IE Resident Inspector	W. Guldemond
20.	Licensing Project Manager	C. M. Trammell
21.	Docket Number	50-301
22.	License Number/Date of Issue	DPR-27 issued 11/18/71
23.	Public Document Room	Joseph Mann Public Library Two Rivers, Wisconsin Manitowoc Public Library Manitowoc, Wisconsin (Temporary LPDR for spent fuel pool capacity increase amendment only

INSPECTION STATUS

ENFORCEMENT STATUS

TECHNICAL SPECIFICATIONS 15.6.8.1 STATES IN PART THAT THE PLANT SHALL BE OPERATED AND MAINTAINED IN ACCORDANCE WITH APPROVED PROCEDURES. RADIOLOGICAL CONTROL PROCEDURES ARE PROMULGATED THROUGH THE POINT BEACH NUCLEAR PLANT HEALTH PHYSICS ADMINISTRATIVE CONTROL POLICIES AND PROCEDURES MANUAL. (A) PART 2.1 OF THE MANUAL STATES IN PART THAT NORMAL ENTRY TO AND EXIT FROM THE CONTROLLED ZONE SHALL BE THROUGH THE TWO DESIGNATED ACCESS CONTROL POINTS ONLY. PART 2.5 STATES IN PART THAT PROTECTIVE CLOTHING REQUIREMENTS WITHIN CONTROLLED AREAS ARE ESTABLISHED BY THE CHEMISTRY AND HEALTH PHYSICS GROUP. CONTRARY TO THE ABOVE, ON JULY 1, 1980, A LICENSEE EMPLOYEE WAS OBSERVED ENTERING THE CONTROLLED ZONE AT THE SOUTH SIDE OF THE FACADE VIA A NONDESIGNATED ACCESS CONTROL POINT, NOT WEARING SPECIFIED PROTECTIVE CLOTHING, AND REMOVING MATERIAL FROM THE CONTROLLED AREA; AND (B) PART 2.4 OF THE MANUAL STATES IN PART THAT ALL RADIOACTIVE MATERIAL OUTSIDE CONTROLLED AREAS ARE TO BE TAGGED WITH A PROPERLY EXECUTED RADIOACTIVE MATERIAL TAG AND EITHER PACKAGED OR SEALED TO PREVENT THE RELEASE OF CONTAMINATION. CONTRARY TO THE ABOVE, ON JULY 2, 1980, AN OPEN TANK LABELLED "RADIOACTIVE MATERIAL" AND CONTAINING LOW LEVEL CONTAMINATION WAS LOCATED IN THE TURBINE BUILDING OUTSIDE OF A CONTROLLED AREA WITHOUT HAVING A PROPERLY EXECUTED RADIOACTIVE MATERIAL TAG. (8012 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

The unit operated routinely during the month.

LAST IE SITE INSPECTION DATE: July 17-22, 1980

INSPECTION REPORT NO: 80-13

SUMMARY

INSPECTION SUMMARY

+ No inspections conducted.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

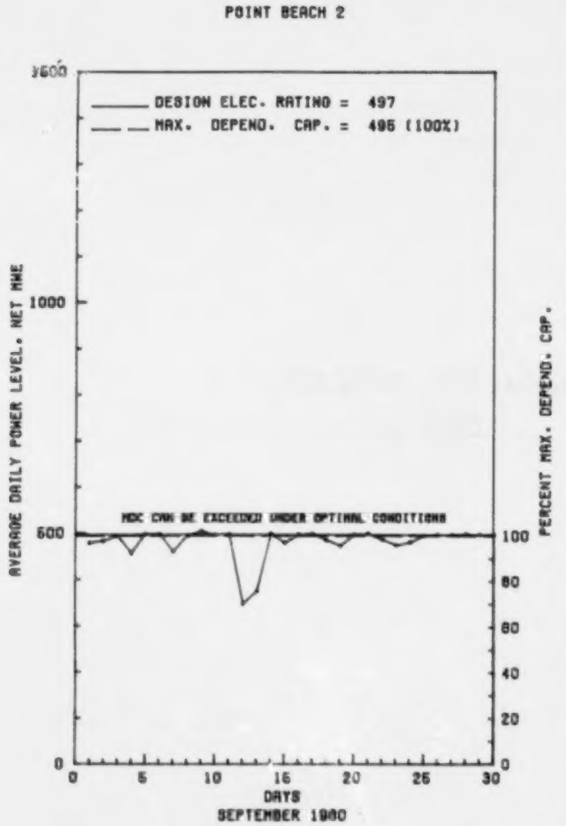
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-008/01T-0	09/12/80	09/26/80	"B" MAIN STEAM STOP VALVE FAILED TO CLOSE UPON RECEIVING CLOSE SIGNAL INITIATED BY LOW T_{AVG} WITH HIGH STEAM FLOW IND. AND SAFETY INJECTION SIGNAL FOLLOWING R_x TRIP. TRIP OCCURRED DUE TO ROD DROP CAUSING LOW PRESSURE. CAUSE: ATWOOD SWING CHECK FAILED TO CLOSE.

REVIEWED BY OMPA LOEB: DATE: 11/05/80

REVIEWED BY NRR DL: DATE: 11/06/80

POINT BEACH 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-301
2. Utility Contact: C.W. FAY (414) 277-2811
3. Licensed Thermal Power (MWt): 1518
4. Nameplate Rating (Gross MWe): 582 x 0.9 = 524
5. Design Electrical Rating (Net MWe): 497
6. Maximum Dependable Capacity (Gross MWe): 519
7. Maximum Dependable Capacity (Net MWe): 495
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

ITEM 21 CALCULATED BY USING A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	6,575.0	71,592.0
12. Hours Reactor Was Critical	711.2	5,466.1	64,363.0
13. Reactor Reserve Shutdown Hours	.4	19.6	185.7
14. Hours Generator On-Line	709.8	5,398.5	63,125.5
15. Unit Reserve Shutdown Hours	1.8	15.3	121.7
16. Gross Thermal Energy (MWH)	1,057,595	7,920,965	85,573,581
17. Gross Electrical Energy (MWH)	362,820	2,661,470	29,022,810
18. Net Electrical Energy (MWH)	346,359	2,535,105	27,610,232
19. Unit Service Factor	98.6	82.1	88.2
20. Unit Availability Factor	98.8	82.3	88.3
21. Unit Capacity Factor (MDC Net)	97.2	77.9	78.6
22. Unit Capacity Factor (DER Net)	96.8	77.6	77.6
23. Unit Forced Outage Rate	1.4	6.3	1.9

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
5	09/12/80	F	10.2	A	3		IF	INSTRU	A FAILED ROD DRIVE POWER SUPPLY CAUSED A GROUP OF RODS TO DROP. THE UNIT TRIPPED DUE TO PRIMARY SYSTEM PRESSURE. THE POWER SUPPLY WAS REPLACED.

SUMMARY

POINT BEACH 2 OPERATED AT OR NEAR FULL POWER FOR THE REPORTING PERIOD, WITH ONE OUTAGE ON THE 12TH CAUSED BY A CONTROL ROD POWER SUPPLY FAILURE.

LEGEND

[1] Type:
F–Forced
S–Scheduled

[2] Reason:
A–Equipment Failure
B–Maintenance or Test
C–Refueling
D–Regulatory Restrictio
E–Operator Training & License Examination
F–Administrative
G–Operational Error
H–Other

[3] Method:
1–Manual
2–Manual Scram
3–Automatic Scram
4–Continuations
5–Load Reduction
9–Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG 0161)

[5] Exhibit H: Same source.

PRAIRIE ISLAND 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Minnesota/Goodhue 28 miles SE of Minneapolis
2.	Electric Reliability Council	Mid-Continent Area Reliability Coordination Agreement
3.	Type of Reactor	PWR
4.	Power Level (MWT)	1650
5.	Design Electrical Rating (MWE Net)	530
6.	Date of Initial Criticality	December 1, 1973
7.	Date of Commercial Operation	December 16, 1973
8.	Condenser Cooling Method	Cooling Towers
9.	Condenser Cooling Water Source	Mississippi River

Utility & Contractor Information

10.	Licensee	Northern States Power Company
11.	Corporate Address	414 Nicollet Mall - 8th Floor Minneapolis, Minnesota 55401
12.	Corporate Contact	G. H. Neils (Oper.) L. O. Mayer (Lic.)
13.	Architect/Engineer	Pioneer Services & Eng. Co.
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Northern States Power Co.

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	W. S. Little
19.	IE Resident Inspector	C. D. Feierabend 612-388-8209
20.	Licensing Project Manager	R. Martin
21.	Docket Number	50-282
22.	License Number/Date of Issue	DPR-42 issued 8/9/73
23.	Public Document Room	Environmental Conservation Library Minneapolis Public Library 300 Nicollet Mall Minneapolis, Minnesota 55401

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

+ Refueling outage.

+LAST IE SITE INSPECTION DATE: August 1-31, 1980

+ INSPECTION REPORT NO: 80-14

SUMMARY

INSPECTION SUMMARY

+ Inspection on July 23 and September 8-9, (80-12): QA/QC program, implementing procedures, work activities, and review of nondestructive examination (NDE) data of inservice inspection (ISI); IE Bulletin 79-03A, IE Bulletin 79-17 and IE Bulletin 80-08 activities. The inspection involved 13 inspector-hours onsite and 3 inspector-hours at Westinghouse Electric Corporation by one NRC inspector. No items of noncompliance or deviations were identified.

+ Inspection on August 1-31, (80-14): Routine resident inspection of plant operation, maintenance, surveillance, security, radiation protection, organization, training and followup on IE Bulletins. The inspection involved 89 inspector-hours onsite by two NRC inspectors including 15 hours during off-shift inspection. No items of noncompliance were identified.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-13/ 03L-0	04/11/80	05/12/80	While performing maintenance a workman accidentally tripped overspeed trip valve for feedwater pump.
80-14/ 03L-0	04/10/80	05/09/80	During surveillance test, one overpower delta T setpoint was found to trip about 70% above the desired setpoint.
80-21/ 03L-0	07/18/80	08/15/80	Design change discrepancy.
80-22/ 03L-0	08/05/80	09/02/80	Inoperable fan coil unit dome damper.
80-23/ 03L-0	08/20/80	09/19/80	Missed SP-1185 control room special vent flow verification.
80-25/ 01T-0	09/04/80 revised 09/25/80	09/18/80	Steam generator bolting found defective.
80-03		09/25/80	Chlorine (Environmental Report).

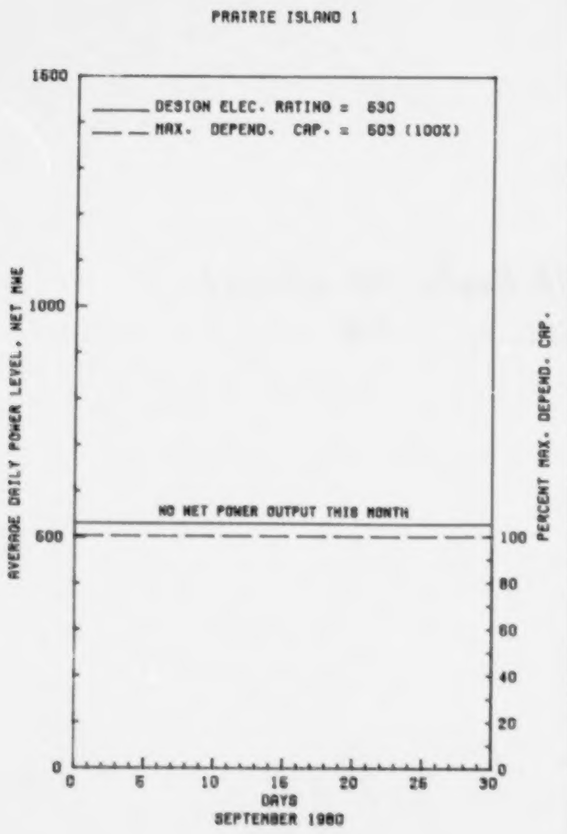
==

REVIEWED BY OMPA LOEB: DmR Beebe **DATE:** 11/05/80

REVIEWED BY NRR DL: R E Martin **DATE:** 11/5/80

PRAIRIE ISLAND 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-282
2. Utility Contact: A. A. Hunstad (612) 388-4767
3. Licensed Thermal Power (MWt): 1650
4. Nameplate Rating (Gross MWe): 659 x 0.9 = 593
5. Design Electrical Rating (Net MWe): 530
6. Maximum Dependable Capacity (Gross MWe): 534
7. Maximum Dependable Capacity (Net MWe): 503
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	6,575.0	59,543.0
12. Hours Reactor Was Critical	.0	5,756.4	46,464.8
13. Reactor Reserve Shutdown Hours	.0	.0	5,531.9
14. Hours Generator On-Line	.0	5,246.5	45,370.1
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	8,140,749	70,219,743
17. Gross Electrical Energy (MWH)	0	2,493,680	22,447,350
18. Net Electrical Energy (MWH)	0	2,327,064	21,166,316
19. Unit Service Factor	.0	79.8	76.2
20. Unit Availability Factor	.0	79.8	76.2
21. Unit Capacity Factor (MDC Net)	.0	70.4	70.7
22. Unit Capacity Factor (DER Net)	.0	66.8	67.1
23. Unit Forced Outage Rate	.0	3.2	12.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
REFUELING - SEPTEMBER 1981 - 6 WEEKS
25. If Shut Down At End of Report Period, Estimated Date of Startup: 10/20/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	08/31/80	S	720.0	C	4				REFUELING OUTAGE CONTINUES.

SUMMARY

PRAIRIE ISLAND 1 REMAINED OFF-LINE FOR SEPT. DUE TO REFUELING.

LEGEND

[1] Type:
F-Forced
S-Scheduled

[2] Reason:
A-Equipment Failure
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error
H-Other

[3] Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Continuations
5-Load Reduction
9-Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

PRAIRIE ISLAND 2

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Minnesota/Goodhue 28 miles SE of Minneapolis
2.	Electric Reliability Council	Mid-Continent Area Reliability Coordination Agreement
3.	Type of Reactor	PWR
4.	Power Level (MWT)	1650
5.	Design Electrical Rating (MWE Net)	530
6.	Date of Initial Criticality	December 17, 1974
7.	Date of Commercial Operation	December 21, 1974
8.	Condenser Cooling Method	Cooling Towers
9.	Condenser Cooling Water Source	Mississippi River

Utility & Contractor Information

10.	Licensee	Northern States Power Company
11.	Corporate Address	414 Nicollet Mall - 8th Floor Minneapolis, Minnesota 55401
12.	Corporate Contact	G. H. Neils (Oper.) L. O. Mayer (Lic.)
13.	Architect/Engineer	Pioneer Services & Eng. Co.
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Northern States Power Co.

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	W. S. Little
19.	IE Resident Inspector	C. D. Feierabend 612-388-8209
20.	Licensing Project Manager	R. Martin
21.	Docket Number	50-306
22.	License Number/Date of Issue	DPR-60 issued 10/29/74
23.	Public Document Room	Environmental Conservation Library Minneapolis Public Library 300 Nicollet Mall Minneapolis, Minnesota 55401

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

Operating routinely.

+LAST IE SITE INSPECTION DATE: August 1-31, 1980

+ INSPECTION REPORT NO: 80-15

SUMMARY

INSPECTION SUMMARY

+ Inspection on July 23 and September 8-9, (80-12): QA/QC program, implementing procedures, work activities, and review of nondestructive examination (NDE) data of inservice inspection (ISI); IE Bulletin 79-03A, IE Bulletin 79-17 and IE Bulletin 80-08 activities. The inspection involved 13 inspector-hours onsite and 3 inspector-hours at Westinghouse Electric Corporation by one NRC inspector. No items of noncompliance or deviations were identified.

+ Inspection on September 3-5, (80-14): Nuclear material control and accountability requirements of module 85102. The seven areas inspected were facility organization and operation, measurements and controls, shipping and receiving, storage and internal control inventory, records and reports, and management of material control system. The inspection involved 40 inspector-hours onsite by two NRC inspectors and was begun during the regular hours. The licensee was found to be in compliance with NRC requirements in the seven areas examined during the inspection.

+ Inspection on August 1-31, (80-15): Routine resident inspection of plant operation, maintenance, surveillance, security, radiation protection, organization, training and followup on IE Bulletins. The inspection involved 89 inspector-hours onsite by two NRC inspectors including 15 hours during off-shift inspection. No items of noncompliance were identified.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

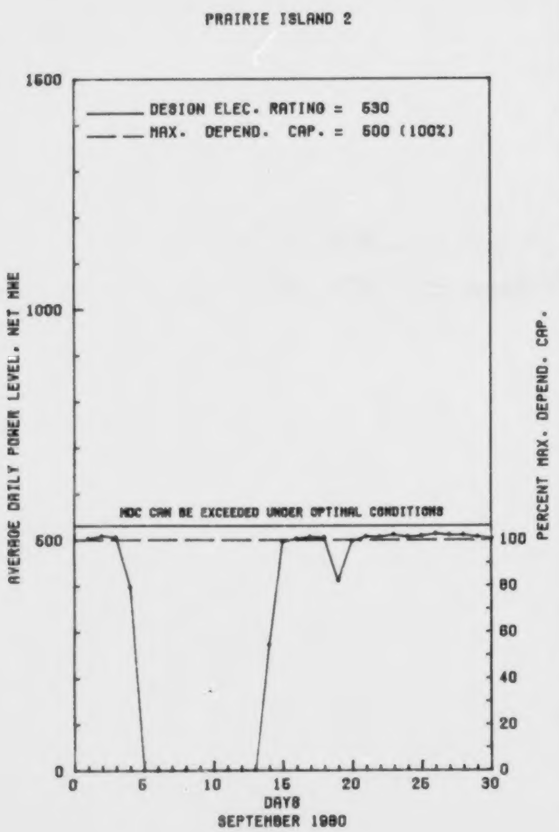
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE			

REVIEWED BY OMPA LOEB: MK Beebe **DATE:** 11/05/80

REVIEWED BY NRR DL: R. Martin **DATE:** 11/05/80

PRAIRIE ISLAND 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-306
2. Utility Contact: A. A. Hunstad (612) 388-4767
3. Licensed Thermal Power (MWt): 1650
4. Nameplate Rating (Gross MWe): 659 x 0.9 = 593
5. Design Electrical Rating (Net MWe): 530
6. Maximum Dependable Capacity (Gross MWe): 531
7. Maximum Dependable Capacity (Net MWe): 500
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	6,575.0	50,661.0
12. Hours Reactor Was Critical	510.3	5,056.4	46,135.4
13. Reactor Reserve Shutdown Hours	.0	89.4	1,495.4
14. Hours Generator On-Line	503.9	4,978.4	43,379.4
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	798,260	7,870,392	67,542,695
17. Gross Electrical Energy (MWH)	261,260	2,535,730	21,587,860
18. Net Electrical Energy (MWH)	242,891	2,368,742	20,207,374
19. Unit Service Factor	70.0	75.7	85.6
20. Unit Availability Factor	70.0	75.7	85.6
21. Unit Capacity Factor (MDC Net)	67.5	72.1	79.8
22. Unit Capacity Factor (DER Net)	63.7	68.0	75.3
23. Unit Forced Outage Rate	.0	.5	4.3

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
REFUELING - JANUARY 1981 - 6 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	09/04/80	S	216.1	B	2	80-25	CC	XXXXXX	SHUTDOWN TO INSPECT STEAM GENERATOR SUPPORT BOLTS.

SUMMARY

PRAIRIE ISLAND 2 OPERATED NORMALLY DURING SEPTEMBER, WITH ONE OUTAGE COMMENCING ON THE 4TH TO INSPECT STEAM GENERATOR SUPPORT BOLTS.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

QUAD CITIES 1

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Illinois/Rock Island 20 miles NE of Moline
2.	Electric Reliability Council	Mid-America Interpool Network
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2511
5.	Design Electrical Rating (MWE Net)	789
6.	Date of Initial Criticality	October 18, 1971
7.	Date of Commercial Operation	February 18, 1973
8.	Condenser Cooling Method	Spray Canal
9.	Condenser Cooling Water Source	Mississippi River

Utility & Contractor Information

10.	Licensee	Commonwealth Edison Co.
11.	Corporate Address	P. O. Box 767 Chicago, Illinois 60690
12.	Corporate Contact	Cordell Reed, Vice President
13.	Architect/Engineer	Sergent and Lundy
14.	Nuclear Steam Supply System	GE
15.	Constructor	United Engineers & Constructors, Inc.

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	R. Spessard
19.	IE Resident Inspector	N. J. Chrissotimos (309) 654-2227
20.	Licensing Project Manager	R. B. Bevan
21.	Docket Number	50-254
22.	License Number/Date of Issue	DPR-29 issued 10/1/71
23.	Public Document Room	Moline Public Library 504 17th Street Moline, Illinois 61265

INSPECTION STATUS

ENFORCEMENT STATUS

AS REQUIRED BY 10 CFR 71.5, SHIPMENTS OF LICENSED RADIOACTIVE MATERIALS MUST COMPLY WITH THE REQUIREMENTS OF THE DEPARTMENT OF TRANSPORTATION, 49 CFR 170-189. 49 CFR 173.392(C)(1) REQUIRES THAT LOW SPECIFIC ACTIVITY (LSA) MATERIALS BE PACKAGED IN STRONG, TIGHT PACKAGES. CONTRARY TO THE ABOVE, THE TOP OF ONE BIN, NUMBER 5-22-80-001, WAS NOT SECURELY ATTACHED TO THE BIN SO THAT IT COULD NOT BE CONSIDERED A STRONG, TIGHT PACKAGE.
(8017 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

The unit was shut down on August 31, 1980 to begin a scheduled 12 week refueling outage.

+LAST IE SITE INSPECTION DATE: August 28, 1980

+ INSPECTION REPORT NO: 80-21

SUMMARY

INSPECTION SUMMARY

+ Inspection on August 28, (80-21): Confirmatory Measurements including: telephone discussion and closeout of results of a previous sample collection. The inspection involved 0 inspector-hours onsite by one NRC inspector. No items of noncompliance or deviations were identified.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-19/ 03L-0	08/18/80	08/25/80	With Unit 1 operating at 400 MWe, RCIC monthly pump oper. surv. was performed and at conclusion the flow controller was left in manual at a reduced flow.

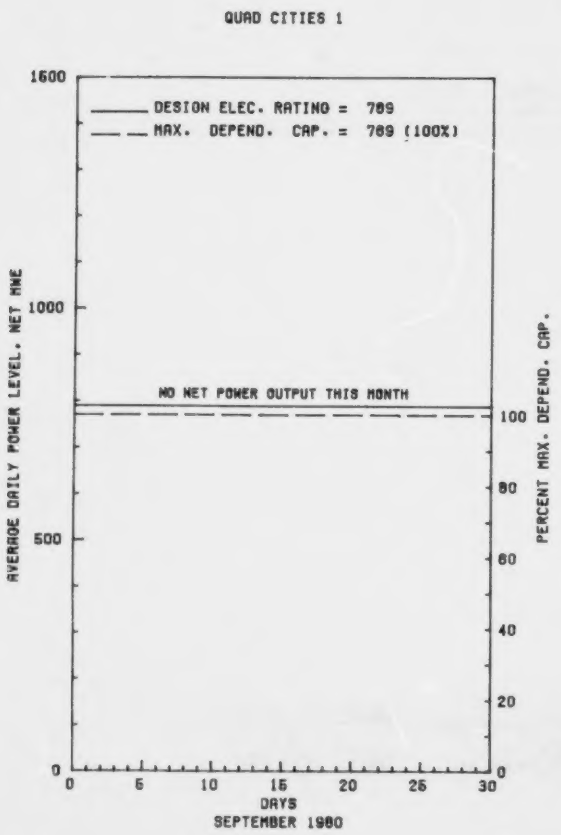
==

REVIEWED BY OMPA LOEB: DATE: 11-4-80

REVIEWED BY NRR DL: RBBevan DATE: 11/6/80

QUAD CITIES 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-254
2. Utility Contact: R. TUBBS (309) 654-2241 X176
3. Licensed Thermal Power (MWt): 2511
4. Nameplate Rating (Gross MWe): 920 x 0.9 = 828
5. Design Electrical Rating (Net MWe): 789
6. Maximum Dependable Capacity (Gross MWe): 813
7. Maximum Dependable Capacity (Net MWe): 769
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	6,575.0	[illegible],535.0
12. Hours Reactor Was Critical	.0	5,725.1	60,439.2
13. Reactor Reserve Shutdown Hours	.0	.0	3,421.9
14. Hours Generator On-Line	.0	5,623.1	57,664.0
15. Unit Reserve Shutdown Hours	.0	.0	909.2
16. Gross Thermal Energy (MWH)	0	11,334,718	115,938,699
17. Gross Electrical Energy (MWH)	28	3,662,758	37,265,812
18. Net Electrical Energy (MWH)	-15,163	3,370,446	34,785,982
19. Unit Service Factor	.0	85.5	78.4
20. Unit Availability Factor	.0	85.5	79.7
21. Unit Capacity Factor (MDC Net)	.0	66.7	61.5
22. Unit Capacity Factor (DER Net)	.0	65.0	60.0
23. Unit Forced Outage Rate	.0	3.6	7.4

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 11/22/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
13	08/31/80	S	720.0	C	4		RC	FUELXX	CONTINUATION OF END OF CYCLE FIVE REFUELING OUTAGE.

SUMMARY

QUAD CITIES 1 REMAINED OFF-LINE FOR THE ENTIRE MONTH IN A CONTINUATION OF CYCLE 5 REFUELING OUTAGE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

QUAD CITIES 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1. Location (state/county) (nearest population center)	Illinois/Rock Island 20 miles NE of Moline
2. Electric Reliability Council	Mid-America Interpool Network
3. Type of Reactor	BWR
4. Power Level (MWT)	2511
5. Design Electrical Rating (MWE Net)	789
6. Date of Initial Criticality	April 26, 1972
7. Date of Commercial Operation	March 10, 1973
8. Condenser Cooling Method	Spray Canal
9. Condenser Cooling Water Source	Mississippi River

Utility & Contractor Information

10. Licensee	Commonwealth Edison Co.
11. Corporate Address	P. O. Box 767 Chicago, Illinois 60690
12. Corporate Contact	Cordell Reed, Vice President
13. Architect/Engineer	Sargent & Lundy
14. Nuclear Steam Supply System	GE
15. Constructor	United Engineers & Constructors, Inc.

Regulatory Information

16. IE Region responsible	III
17. IE Branch Chief	R. Heishman
18. IE Section Chief	R. Spessard
19. IE Resident Inspector	N. J. Chrissotimos (309) 654-2227
20. Licensing Project Manager	R. B. Bevan
21. Docket Number	50-265
22. License Number/Date of Issue	DPR-30 issued 3/21/72
23. Public Document Room	Moline Public Library 504 17th Street Moline, Illinois 61265

INSPECTION STATUS

ENFORCEMENT STATUS

AS REQUIRED BY 10 CFR 71.5, SHIPMENTS OF LICENSED RADIOACTIVE MATERIALS MUST COMPLY WITH THE REQUIREMENTS OF THE DEPARTMENT OF TRANSPORTATION, 49 CFR 1 70-189. 49 CFR 173.392(C)(1) REQUIRES THAT LOW SPECIFIC ACTIVITY (LSA) MATERIALS BE PACKAGED IN STRONG, TIGHT PACKAGES. CONTRARY TO THE ABOVE, THE TOP OF ONE BIN, NUMBER 5-22-80-001, WAS NOT SECURELY ATTACHED TO THE BIN SO THAT IT COULD NOT BE CONSIDERED A STRONG, TIGHT PACKAGE.
(8019 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

Plant operating routinely.

+LAST IE SITE INSPECTION DATE: August 28, 1980

+ INSPECTION REPORT NO: 80-23

SUMMARY

INSPECTION SUMMARY

+ Inspection on August 28, (80-23): Confirmatory Measurements including: telephone discussion and closeout of results of a previous sample collection. The inspection involved 0 inspector-hours onsite by one NRC inspector. No items of noncompliance or deviations were identified.

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-18/01T-0	08/29/80	09/08/80	While performing HPCI operability test, QOS 2300-2, it was discovered that the HPCI motor gear unit would not come off the high speed stop.
80-19/03L-0	08/29/80	09/22/80	The "A" recirculation pump motor generator field breaker opened.
80-20/03L-0	08/28/80	09/19/80	While performing the Suppression Chamber to Drywell Separation Test, QOS 1600-3, nitrogen make-up valve MO2-1601-57 would not close.

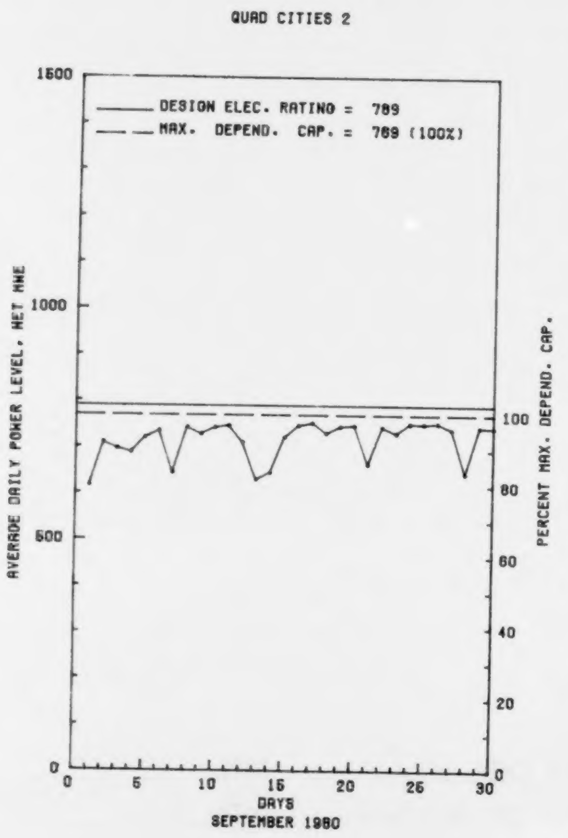
==

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 11-4-80

REVIEWED BY NRR DL: RBBevan **DATE:** 11/6/80

QUAD CITIES 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-265
2. Utility Contact: ROBERT TUBBS (309) 654-2241 X176
3. Licensed Thermal Power (MWt): 2511
4. Nameplate Rating (Gross MWe): 920 x 0.9 = 828
5. Design Electrical Rating (Net MWe): 789
6. Maximum Dependable Capacity (Gross MWe) 813
7. Maximum Dependable Capacity (Net MWe): 769
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	6,575.0	72,645.0
12. Hours Reactor Was Critical	720.0	3,781.5	56,910.4
13. Reactor Reserve Shutdown Hours	.0	.0	2,985.8
14. Hours Generator On-Line	720.0	3,662.0	54,453.8
15. Unit Reserve Shutdown Hours	.0	.0	702.9
16. Gross Thermal Energy (MWH)	1,723,851	8,178,905	111,195,014
17. Gross Electrical Energy (MWH)	539,384	2,557,070	35,420,120
18. Net Electrical Energy (MWH)	514,940	2,395,916	33,138,441
19. Unit Service Factor	100.0	55.7	75.0
20. Unit Availability Factor	100.0	55.7	75.9
21. Unit Capacity Factor (MDC Net)	93.0	47.4	59.3
22. Unit Capacity Factor (DER Net)	90.6	46.2	57.8
23. Unit Forced Outage Rate	.0	5.7	9.1

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	NONE								

SUMMARY

QUAD CITIES 2 OPERATED AT APPROXIMATELY 90% POWER FOR SEPTEMBER, WITH NO OUTAGES OR REDUCTIONS.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

RANCHO SECO

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	California/Sacramento 25 miles SE of Sacramento
2.	Electric Reliability Council	Western Systems Coordinating Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2772
5.	Design Electrical Rating (MWE Net)	918
6.	Date of Initial Criticality	September 16, 1974
7.	Date of Commercial Operation	April 17, 1975 (92.6% of full power)
8.	Condenser Cooling Method	Closed Cycle
9.	Condenser Cooling Water Source	Natural Draft Cooling Towers - Folsom Canal

Utility & Contractor Information

10.	Licensee	Sacramento Municipal Utility District
11.	Corporate Address	6201 South Street P. O. Box 15830 Sacramento, California
12.	Corporate Contact	J. J. Mattimoe Assistant General Manager, Chief Engineer
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	B&W
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	V
17.	IE Branch Chief	J. L. Crews
18.	IE Section Chief	B. H. Faulkenberry
19.	IE Resident Inspector	H. L. Canter (209) 748-2791
20.	Licensing Project Manager	M. L. Padovan
21.	Docket Number	50-312
22.	License Number/Date of Issue	DPR-54, 8/16/74
23.	Public Document Room	Business and Municipal Department Sacramento City-County Library 828 I Street Sacramento, California 95814

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO SECTION 12.3.2 OF THE FSAR, THE LICENSEE DID NOT MAINTAIN FIVE FULL STRENGTH SHIFT CREWS SO THAT RELIEF CREW COULD RECEIVE TRAINING WHILE REGULARLY ASSIGNED TO DAY SHIFT DURING 1979.
(8021 DEVIATION)

CONTRARY TO ANSI N18.7-1972, SECTION 4.4, FAILED TO REVIEW WRITTEN REPORTS OF AUDITS AT A SCHEDULED MEETING OF THE INDEPENDENT REVIEW AND AUDIT GROUP (MSRC).
(8024 DEVIATION)

CONTRARY TO TECHNICAL SPECIFICATION, SECTIONS 6.5.2.8.A AND 6.5.2.10 THERE WERE NO DOCUMENTED AUDITS DEMONSTRATING CONFORMANCE OF FACILITY OPERATION TO THE LSSS AND LCO PROVISIONS OF THE TECHNICAL SPECIFICATIONS AT LEAST ONCE A YEAR.
(8024 INFRACTION)

INSPECTION STATUS

SYSTEMS AND COMPONENT PROBLEMS:

PROBLEM WITH TURBINE AND COMPONENT COOLING WATER SYSTEM CAUSED APPROXIMATELY THREE WEEK OUTAGE DURING REPORT PERIOD.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

+ AWAITING DECISION OF ASLB. HEALTH PHYSICS APPRAISAL TEAM INSPECTION SEPTEMBER 27 - OCTOBER 3, 1980.

PLANT STATUS:

+ OPERATION AT OR NEAR FULL POWER SINCE SEPTEMBER 4, 1980 FOLLOWING REPAIR OF MAIN TURBINE THRUST BEARING.

LAST IE SITE INSPECTION DATE: 09/29 - 10/03/80+

INSPECTION REPORT NO. 50-312/80-27+

SUMMARY

INSPECTION SUMMARY

+ INSPECTION ON AUGUST 1-29, 1980 (REPORT NO. 50-312/80-26)
AREAS INSPECTED: ROUTINE OPERATIONS SAFETY VERIFICATION; MONTHLY MAINTENANCE OBSERVATION; MONTHLY SURVEILLANCE OBSERVATION; LER FOLLOWUP, FOLLOWUP ON REGIONAL REQUEST; FOLLOWUP ON HEADQUARTERS' REQUESTS; AND INDEPENDENT INSPECTION EFFORT. THE INSPECTION INVOLVED 117 INSPECTOR HOURS ONSITE BY TWO NRC RESIDENT INSPECTORS.

RESULTS: OF THE SEVEN AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN FIVE AREAS. TWO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED IN ONE AREA (REPEAT INFRACTION - FAILURE TO RESET OVERPOWER HIGH FLUX TRIP TO 5%; INFRACTION - FAILURE TO FOLLOW SHUTDOWN PROCEDURE B.4), AND ONE INFRACTION WAS IDENTIFIED IN ANOTHER AREA (CONTAINMENT ISOLATION VALVE DESIGN NOT IN ACCORDANCE WITH 10 CFR 50, APPENDIX A REQUIREMENTS).

+ INSPECTION ON SEPTEMBER 29 - OCTOBER 3, 1980
(REPORT NO. 50-312/80-27)
REPORT BEING PREPARED; TO BE REPORTED NEXT MONTH.

+ INSPECTION ON SEPTEMBER 2-5, 1980 (REPORT NO. 50-312/80-28)
AREAS INSPECTED: ROUTINE, ANNOUNCED INSPECTION BY A REGIONAL BASED INSPECTOR OF THE LICENSEE'S ACTIVITIES PERFORMED IN RESPONSE TO IE BULLETIN 79-01B, "ENVIRONMENTAL QUALIFICATION OF CLASS 1E EQUIPMENT." THE INSPECTION INVOLVED 26 INSPECTOR HOURS ONSITE AND AT SMUD HEADQUARTERS BY ONE NRC INSPECTOR.

RESULTS: NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

NRC POWER RESTRICTIONS: None

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-39/ 01T-0	08-13-80	09-02-80	DID NOT RESET HIGH FLUX TRIP TO 5 PCT WHILE IN SHUTDOWN BYPASS.
80-40/ 03L-0	08-12-80	09-12-80	DEFECTIVE FLOW XMITTER IN RPS-BAILEY BY-3X 41XA RPS FLOW XMTR.
80-41/ 03L-0	08-23-80	09-22-80	DIESEL GEN B VOLTAGE WAS 4100V INSTEAD OF 4160V.

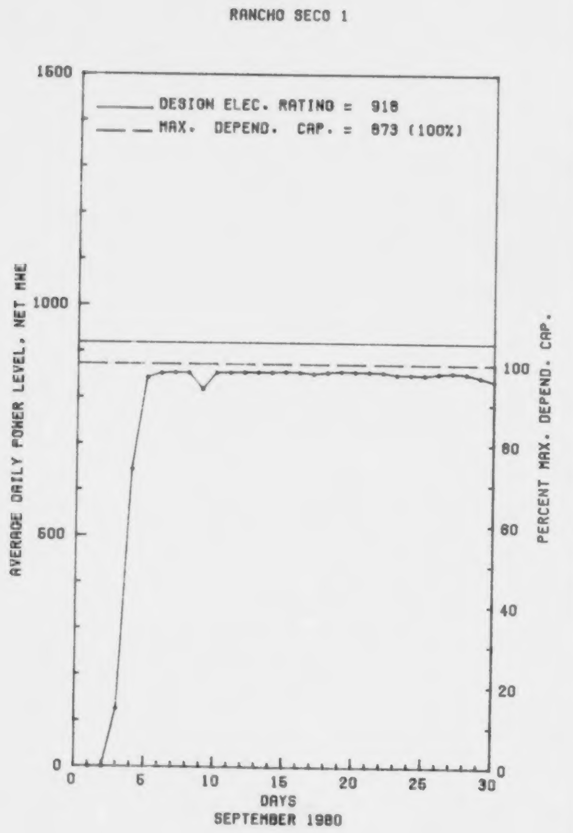
+ INSPECTION ON SEPTEMBER 2-30, 1980 (REPORT NO. 50-312/80-29)
REPORT BEING PREPARED; TO BE REPORTED NEXT MONTH.

REVIEWED BY OMPA LOEB: PE Boise **DATE:** 11/5/80

REVIEWED BY NRR DL: M L Padovan **DATE:** 11/5/80

RANCHO SECO

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-312
2. Utility Contact: R. HOLLINGSWORTH (916) 452-3211
3. Licensed Thermal Power (MWt): 2772
4. Nameplate Rating (Gross MWe): 1070 x 0.9 = 963
5. Design Electrical Rating (Net MWe): 918
6. Maximum Dependable Capacity (Gross MWe): 917
7. Maximum Dependable Capacity (Net MWe): 873
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	6,575.0	47,832.0
12. Hours Reactor Was Critical	667.9	3,216.4	29,289.9
13. Reactor Reserve Shutdown Hours	52.1	494.5	4,469.6
14. Hours Generator On-Line	664.5	3,105.9	8,247.2
15. Unit Reserve Shutdown Hours	.0	.0	1,210.2
16. Gross Thermal Energy (MWH)	1,738,460	8,020,164	71,053,862
17. Gross Electrical Energy (MWH)	571,186	2,657,486	2[illegible],952,022
18. Net Electrical Energy (MWH)	540,303	2,514,212	22,625,102
19. Unit Service Factor	91.7	47.2	56.6
20. Unit Availability Factor	91.7	47.2	61.2
21. Unit Capacity Factor (MDC Net)	86.0	43.8	54.2
22. Unit Capacity Factor (DER Net)	81.7	41.7	51.5
23. Unit Forced Outage Rate	8.3	16.5	30.7

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUELING, APRIL - JUNE 1981
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
7	08/12/80	F	59.5	A	4		HA		TURBIN OUTAGE CAUSED BY DAMAGED THRUST BEARING (CONTINUED).

SUMMARY

RANCHO SECO OPERATED NEAR FULL POWER FOR THE MONTH, AFTER RETURNING ON-LINE ON THE 3RD FROM AN OUTAGE TO REPAIR A THRUST BEARING.

LEGEND

[1] Type:
F-Forced
S-Scheduled

[2] Reason:
A-Equipment Failure
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error
H-Other

[3] Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Continuations
5-Load Reduction
9-Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

ROBINSON 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	South Carolina/Darlington 5miles NW of Hartsville
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2300
5.	Design Electrical Rating (MWE Net)	700
6.	Date of Initial Criticality	September 20, 1970
7.	Date of Commercial Operation	March 7, 1971
8.	Condenser Cooling Method	Recirculation from impounded lake
9.	Condenser Cooling Water Source	Robinson Impoundment

Utility & Contractor Information

10.	Licensee	Carolina Power and Light Company
11.	Corporate Address	411 Fayetteville Street Raleigh, North Carolina
12.	Corporate Contact	J. A. Jones Executive Vice President
13.	Architect/Engineer	Ebasco
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Ebasco

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R.C. Lewis
18.	IE Section Chief	P. J. Kellogg
19.	IE Resident Inspector	A. K. Hardin
20.	Licensing Project Manager	D. Neighbors
21.	Docket Number	50-261
22.	License Number/Date of Issue	DPR-23, 7/31/70 (5 Mwt), Full power authorization issued 9/23/70
23.	Public Document Room	Hartsville Memorial Library Home and Fifth Avenue Hartsville, South Carolina 29550

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO TECHNICAL SPECIFICATION 6.5.1.7A THE CHEMICAL AND VOLUME CONTROL SYSTEM VENT HEADER BACKUP PRESSURE SOURCE WAS ISOLATED WITHOUT REVIEW AND APPROVAL.
(8021 INFRACTION)

CONTRARY TO TECHNICAL SPECIFICATION 6.8.1 MAINTENANCE PROCEDURES WERE EITHER NOT ESTABLISHED OR FOLLOWED RESULTING IN FOUR EXAMPLES OF ELECTRICAL OR SEISMIC SUPPORT NONCONFORMANCES.
(8021 INFRACTION)

CONTRARY TO 10 CFR 55 APPENDIX A, THE OPERATOR REQUALIFICATION PROGRAM DID NOT ASSURE OPERATORS WERE COGNIZANT OF FACILITY DESIGN AND PROCEDURE CHANGES.
(8021 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

+ SHUTDOWN FOR REFUELING

+ LAST IE SITE INSPECTION DATE: SEPTEMBER 3-5, 1980

INSPECTION REPORT NO: 50-261/80-23

SUMMARY

INSPECTION SUMMARY

+ INSPECTION JULY 27 - AUGUST 1 (80-14): INSPECTED CHEM-NUCLEAR WASTE DISPOSAL SITE, BARNWELL, SOUTH CAROLINA FOR UNANNOUNCED INSPECTIONS OF LICENSEE-SHIPPERS. EACH LICENSEE-SHIPPER VEHICLE WAS INSPECTED FOR COMPLIANCE WITH DEPARTMENT OF TRANSPORTATION (DOT) AND NUCLEAR REGULATORY COMMISSION (NRC) REGULATIONS AS FOLLOWS: (1) SHIPPING PAPER REQUIREMENTS; (2) DOT PLACARDING REQUIREMENTS; (3) RADIATION LEVELS; (4) REMOVABLE CONTAMINATION; (5) DOT MARKING AND LABELING REQUIREMENTS FOR PACKAGES; (6) DOT AND NRC REQUIREMENTS FOR EXTERNAL PACKAGE FEATURES, AND (7) PROHIBITED ARTICLES OR CONTENTS. OF THE SEVEN AREAS INSPECTED INVOLVING 98 SHIPMENTS, NO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED.

INSPECTION AUGUST 11-14 (80-19): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 30 INSPECTOR-HOURS ON SITE IN THE AREAS OF DIESEL GENERATOR UNITS USED AS ONSITE ELECTRIC POWER SYSTEMS, REVIEW OF IE BULLETIN 80-06, AND REVIEW OF DIESEL GENERATOR TESTING REQUIREMENTS AT IE HEADQUARTERS' REQUEST. OF THE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION AUGUST 25-29 (80-20): THIS SPECIAL, ANNOUNCED INSPECTION INVOLVED 61 INSPECTOR-HOURS ON SITE IN THE AREA OF INSTALLED ELECTRICAL EQUIPMENT REVIEW WITH RESPECT TO THE IE BULLETIN 79-01B RESPONSE. OF THE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION AUGUST 18-22 (80-21): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 40 INSPECTOR-HOURS ON SITE IN THE AREAS OF IE BULLETIN FOLLOWUP AND PLANT TOUR. THREE APPARENT ITEMS OF COMPLIANCE WERE FOUND IN THE TWO AREAS; (INFRACTION - FAILURE TO REVIEW PROCEDURE CHANGE; INFRACTION - FAILURE TO ESTABLISH OR IMPLEMENT MAINTENANCE PROCEDURE; AND DEFICIENCY - FAILURE TO COMPLETE OPERATOR REQUIRED READING).

INSPECTION AUGUST 5-14 (80-22): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 92 (RESIDENT) INSPECTOR-HOURS ON SITE IN THE AREAS OF WITNESSING GENERAL PLANT OPERATIONS, FOLLOWUP INSPECTION OF OUTSTANDING ITEMS, INSPECTION OF HYDRAULIC SUPPORTS AND RESTRAINTS, AND WITNESSING PORTIONS OF THE SAFETY INJECTION TESTS. OF THE 4 AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION SEPTEMBER 3-5 (80-23): THIS ROUTINE ANNOUNCED INSPECTION INVOLVED 24 INSPECTOR-HOURS ON SITE IN THE AREAS OF INSERVICE INSPECTION PROGRAM, PROCEDURES AND RECORDS, IE BULLETIN 79-13 FOLLOWUP INSPECTION, CRD CANOPY REPAIR WORK, PREVIOUSLY IDENTIFIED INSPECTION FINDINGS. OF THE FOUR AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

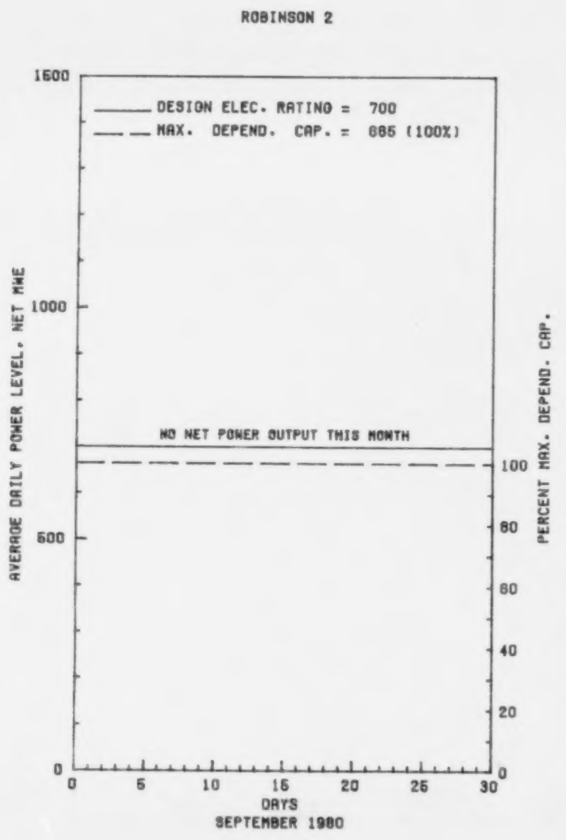
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-017/ 03L-0	07/30/80	08/29/80	DISCHARGE DAMPER ON CONTROL ROOM VENT SYSTEM FAILED TO OPERATE PROPERLY.
80-018/ 03L-0	08/11/80	09/10/80	VALVE PCV-1716 INST. AIR TO CONTAINMENT FAIL TO CLOSE.

REVIEWED BY OMPA LOEB: PE Bobe **DATE:** 11/5/80

REVIEWED BY NRR DL: Don Neighbors **DATE:** 11/6/80

ROBINSON 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-261
2. Utility Contact: M. L. Watford (803) 383-4524
3. Licensed Thermal Power (MWt): 2300
4. Nameplate Rating (Gross MWe): 854 x 0.9 = 769
5. Design Electrical Rating (Net MWe): 700
6. Maximum Dependable Capacity (Gross MWe): 700
7. Maximum Dependable Capacity (Net MWe): 665
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): 2200 MWt
10. Reasons for Restrictions, If Any: EXCESSIVE MOISTURE CARRY-OVER TO H.P. TURBINE.

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	6,575.0	83,957.1
12. Hours Reactor Was Critical	.0	4,306.7	64,513.5
13. Reactor Reserve Shutdown Hours	.0	219.8	939.1
14. Hours Generator On-Line	.0	4,282.6	62,973.5
15. Unit Reserve Shutdown Hours	.0	.0	23.2
16. Gross Thermal Energy (MWH)	0	8,207,853	128,758,182
17. Gross Electrical Energy (MWH)	0	2,605,015	41,541,528
18. Net Electrical Energy (MWH)	0	2,454,997	39,359,199
19. Unit Service Factor	.0	65.1	75.0
20. Unit Availability Factor	.0	65.1	75.0
21. Unit Capacity Factor (MDC Net)	.0	56.1	70.5
22. Unit Capacity Factor (DER Net)	.0	53.3	67.0
23. Unit Forced Outage Rate	.0	19.2	13.6

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 10/16/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
08-01	08/08/80	S	720.0	C	4		RC	FUELXX	CONTINUATION OF MAINTENANCE/REFUELING OUTAGE REPORTED FOR AUGUST.

SUMMARY

ROBINSON 2 REMAINED OFF-LINE FOR SEPTEMBER DUE TO REFUELING AND MAINTENANCE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

SALEM 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	New Jersey/Salem 20 miles S of Wilmington, Del.
2.	Electric Reliability Council	Mid-Atlantic Area Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	3338
5.	Design Electrical Rating (MWE Net)	1090
6.	Date of Initial Criticality	December 11, 1976
7.	Date of Commercial Operation	June 30, 1977
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Delaware River

Utility & Contractor Information

10.	Licensee	Public Service Electric & Gas Co.
11.	Corporate Address	80 Park Place - 15A Newark, New Jersey 07101
12.	Corporate Contact	F. P. Librizzi General Manager - Electric Prod.
13.	Architect/Engineer	PSE&G
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	UE&C

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	IE Section Chief	R. R. Keimig
19.	IE Resident Inspector	L. J. Norrholm (609-935-3850) (FTS 341-3000)
20.	Licensing Project Manager	W. Ross
21.	Docket Number	50-272
22.	License Number/Date of Issue	DPR-70, 8/13/76
23.	Public Document Room	Salem Free Public Library 112 West Broadway Salem, New Jersey 08079

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS:

+ THE UNIT IS IN MODE 5 (COLD SHUTDOWN). RESTART IS SCHEDULED FOR MID-DECEMBER.

+ LAST IE SITE INSPECTION DATE: 9/1-30/80

+ INSPECTION REPORT NO: 50-272/80-21

SUMMARY

INSPECTION SUMMARY

+ 50-272/80-19 - AUG 1 AND AUG 4-8: ROUTINE, UNANNOUNCED INSPECTION BY TWO REGION-BASED INSPECTORS (56 HRS) OF QA PROGRAM IMPLEMENTATION INCLUDING AUDITS; DOCUMENT CONTROL; MAINTENANCE; DESIGN CHANGES AND MODIFICATIONS; SURVEILLANCE, TESTING AND CALIBRATION CONTROL; PROCUREMENT CONTROL; RECEIPT, STORAGE AND HANDLING OF PROCURED ITEMS; RECORDS; TEST AND MEASUREMENT EQUIPMENT; AND LICENSEE ACTION ON PPEVIUUSLY IDENTIFIED INSPECTION FINDINGS. TWO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED: FAILURE TO FOLLOW PROCEDURES FOR RECEIPT, HANDLING AND STORAGE OF SAFETY-RELATED AND "QA REQUIRED" MATERIAL; - FAILURE TO VERIFY ENVIRONMENTAL CONDITIONS DURING CALIBRATION OF ELECTRONIC MEASURING AND TEST EQUIPMENT.

+ 50-272/80-20 - AUG 1-31: ROUTINE INSPECTIONS BY THE RESIDENT INSPECTORS (42 HRS) OF PLANT OPERATIONS INCLUDING TOURS OF THE FACILITY; LOG AND RECORD REVIEWS; REVIEW OF LICENSEE EVENTS; OPERATIONAL SAFETY VERIFICATION; IE BULLETIMS; REVIEW OF PERIODIC REPORTS; FUEL RECEIPT ACTIVITIES; AND, FOLLOWUP ON PREVIOUS INSPECTION ITEMS. ONE ITEM OF NONCOMPLIANCE WAS IDENTIFIED: FAILURE TO FOLLOW PROCEDURES.

REPORTS RECEIVED FROM LICENSEE

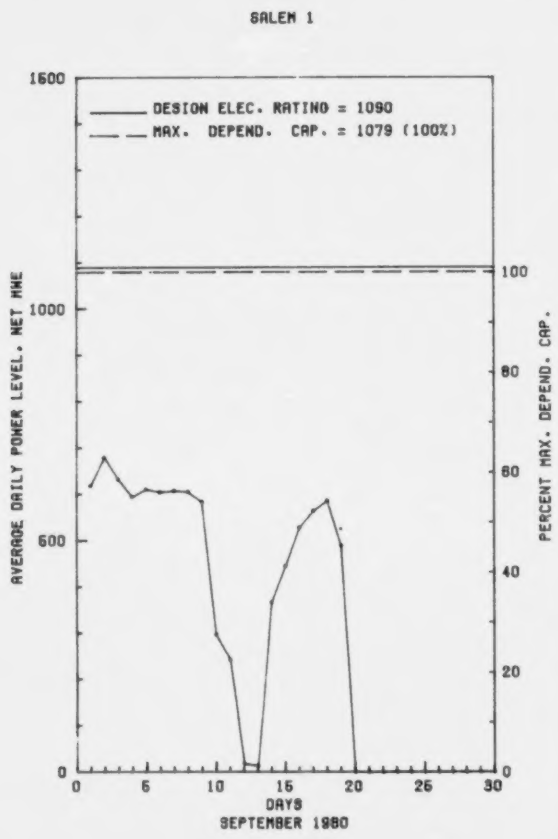
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-043/ 03L	08/01/80	08/28/80	REFUELING WATER STORAGE TANK LEVEL BELOW TECH SPEC REQUIREMENT
80-044/ 03L	08/05/80	09/04/80	CONTAINMENT AIR LOCK INOPERABLE - DOOR ROLLER BROKEN
80-045/ 03L	08/06/80	09/04/80	AIR PARTICULATE DETECTOR INOPERABLE - PUMP SEIZED
80-046/ 03L	08/10/80	09/04/80	STEAM GENERATOR 12 LEVEL FEED-FLOW STEAM-FLOW MISMATCH CIRCUIT FAILED

REVIEWED BY OMPA LOEB: P E Bobe **DATE:** 11/5/80

REVIEWED BY NRR DL: W Ross **DATE:** 11/6/80

SALEM 1

AVERAGE DAILY POWER LEVEL(MWe)



OPERATING STATUS

1. Docket Number: 50-272
2. Utility Contact: L. K. Miller (609) 365-7000
3. Licensed Thermal Power (MWt): 3338
4. Nameplate Rating (Gross MWe): 1300 x 0.9 = 1170
5. Design Electrical Rating (Net MWe): 1090
6. Maximum Dependable Capacity (Gross MWe): 1124
7. Maximum Dependable Capacity (Net MWe): 1079
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	6,575.0	28,536.0
12. Hours Reactor Was Critical	431.9	6,097.8	16,136.1
13. Reactor Reserve Shutdown Hours	.0	.0	22.7
14. Hours Generator On-Line	400.3	5,946.3	15,477.4
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	790,673	18,205,037	45,741,333
17. Gross Electrical Energy (MWH)	236,380	5,935,090	15,102,590
18. Net Electrical Energy (MWH)	213,388	5,664,024	14,293,614
19. Unit Service Factor	55.6	90.4	54.2
20. Unit Availability Factor	55.6	90.4	54.2
21. Unit Capacity Factor (MDC Net)	27.5	79.8	46.4
22. Unit Capacity Factor (DER Net)	27.2	79.0	46.0
23. Unit Forced Outage Rate	12.0	5.8	34.4

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUELING - SEPTEMBER 20, 1980 - DECEMBER 7, 1980.
25. If Shut Down At End of Report Period, Estimated Date of Startup: 12/07/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-187	09/02/80	S	.0	H	5		RC	ZZZZZZ	FUEL DEPLETION
80-189	09/05/80	S	.0	H	5		RC	ZZZZZZ	FUEL DEPLETION
80-191	09/06/80	S	.0	H	5		RC	ZZZZZZ	FUEL DEPLETION
80-193	09/10/80	S	.0	H	5		RC	ZZZZZZ	FUEL DEPLETION
80-195	09/10/80	F	18.0	A	3		HH	VALVOP	LOW LEVEL #12 STEAM GENERATOR DUE TO LOSS OF CONTROL AIR TO VALVE 12BF19.
80-197	09/11/80	S	.0	H	5		RC	ZZZZZZ	FUEL DEPLETION
80-199	09/12/80	F	36.8	H	2		PA	TURBIN	WATER IN TURBINE LUBE OIL.
80-201	09/14/80	S	.0	H	5		RC	ZZZZZZ	FUEL DEPLETION
80-203	09/16/80	S	.0	H	5		RC	ZZZZZZ	FUEL DEPLETION
80-205	09/17/80	S	.0	H	5		RC	ZZZZZZ	FUEL DEPLETION
80-207	09/18/80	S	.0	H	5		RC	ZZZZZZ	FUEL DEPLETION
80-209	09/19/80	S	.0	H	5		RC	ZZZZZZ	FUEL DEPLETION
80-210	09/19/80	S	264.9	C	1		RC	FUELXX	ANNUAL REFUELING OUTAGE.

SUMMARY

SALEM I OPERATED IN A COASTDOWN MODE DURING SEPTEMBER WITH TWO OUTAGES ON THE 10TH AND 12TH FOR EQUIPMENT REPAIR, AND WENT INTO ANNUAL REFUELING ON THE 19TH.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

SAN ONOFRE 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	California/San Diego 5 miles S of San Clemente
2.	Electric Reliability Council	Western Systems Coordinating Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	1347
5.	Design Electrical Rating (MWE Net)	436
6.	Date of Initial Criticality	June 14, 1967
7.	Date of Commercial Operation	January 1, 1968
8.	Condenser Cooling Method	Circulating sea water
9.	Condenser Cooling Water Source	Pacific Ocean

Utility & Contractor Information

10.	Licensee	Southern California Edison Co.
11.	Corporate Address	2244 Walnut Grove Avenue Rosemead, California 91770
12.	Corporate Contact	R. E. Woodbury V.P. & General Counsel
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	V
17.	IE Branch Chief	J. L. Crews
18.	IE Section Chief	B. H. Faulkenberry
19.	IE Resident Inspector	L. F. MILLER (714-492-2641)
20.	Licensing Project Manager	S. J. Nowicki
21.	Docket Number	50-206
22.	License Number/Date of Issue	DPR-13, 3/27/67
23.	Public Document Room	Mission Viejo Branch Library 24851 Chrisanta Drive Mission Viejo, California 92676

INSPECTION STATUS

ENFORCEMENT STATUS

TECHNICAL SPECIFICATION 6.8.1 STATES THAT WRITTEN PROCEDURES AND ADMINISTRATIVE POLICIES SHALL BE ESTABLISHED, IMPLEMENTED AND MAINTAINED THAT MEET OR EXCEED THE REQUIREMENTS AND RECOMMENDATIONS OF APPENDIX A OF USNRC REGULATORY GUIDE 1.33, REVISION I, QUALITY ASSURANCE PROGRAM REQUIREMENTS (OPERATION). APPENDIX A OF THAT GUIDE STATES THAT RADIATION PROTECTION PROCEDURES SHOULD BE WRITTEN, INCLUDING PROCEDURES FOR ACCESS CONTROL TO RADIATION AREAS. STATION RADIATION PROTECTION PROCEDURE, S-VII-1.5, "ACCESS TO CONTROLLED AND EXCLUSION AREAS," STATES THAT RADIATION EXPOSURE PERMITS PROVIDE SPECIFIC RADIATION SAFETY INSTRUCTIONS FOR WORK IN EXCLUSION AREAS. ON JULY 20, 1980, THE INSPECTOR OBSERVED THAT ALL RADIATION EXPOSURE PERMITS IN EFFECT FOR WORK INSIDE THE UNIT I CONTAINMENT BUILDING REQUIRED "GLOVES" AS AN ARTICLE OF PROTECTIVE CLOTHING. CONTRARY TO THE REQUIREMENTS, ON JULY 20, 1980 THE INSPECTOR OBSERVED THAT ONE TECHNICIAN WAS AT WORK ON THE "A" STEAM GENERATOR PLATFORM WITHOUT GLOVES ON SEVERAL OCCASIONS.
(8021 DEFICIENCY)

TECHNICAL SPECIFICATION 6.8.1 STATES THAT WRITTEN PROCEDURES AND ADMINISTRATIVE POLICIES SHALL BE ESTABLISHED, IMPLEMENTED AND MAINTAINED THAT MEET OR EXCEED THE REQUIREMENTS AND RECOMMENDATIONS OF APPENDIX A OF USNRC REGULATORY GUIDE 1.33, REVISION I, QUALITY ASSURANCE PROGRAM REQUIREMENTS (OPERATION). APPENDIX A OF THAT GUIDE STATES THAT RADIATION PROTECTION PROCEDURES SHALL BE WRITTEN, INCLUDING PROCEDURES FOR ACCESS CONTROL TO RADIATION AREAS. STATION RADIATION PROTECTION PROCEDURE, S-VII-1.5, "ACCESS TO CONTROLLED AND EXCLUSION AREAS," STATES THAT EACH PERSON ENTERING AN EXCLUSION AREA MUST WEAR A POCKET DOSIMETER, A LAB COAT OR OTHER PROTECTIVE CLOTHING, AND BE LISTED ON A RADIATION EXPOSURE PERMIT, IN GENERAL. CONTRARY TO THE REQUIREMENTS, ON JULY 21, 1980 THE INSPECTOR OBSERVED THREE LICENSEE PERSONNEL WORKING IN AN EXCLUSION AREA WHO COLLECTIVELY FAILED TO MEET THE APPLICABLE ABOVE CITED REQUIREMENTS OF PROCEDURE S-VII-1.5.
(8021 INFRACTION)

10 CFR 20.101, "RADIATION DOSE STANDARDS FOR INDIVIDUALS IN RESTRICTED AREAS," PARAGRAPH (B) STATES, "A LICENSEE MAY PERMIT AN INDIVIDUAL IN A RESTRICTED AREA TO RECEIVE A TOTAL OCCUPATIONAL DOSE TO THE WHOLE BODY GREATER THAN THAT PERMITTED UNDER PARAGRAPH (A) OF THIS SECTION, PROVIDED: (1) DURING ANY CALENDAR QUARTER THE TOTAL OCCUPATIONAL DOSE TO THE WHOLE BODY SHALL NOT EXCEED 3 REMS..." CONTRARY TO THE REQUIREMENTS, A CONTRACTOR EMPLOYED INDIVIDUAL WHOSE ACCUMULATED OCCUPATIONAL EXPOSURE HISTORY WAS ON FILE, RECEIVED AN EXPOSURE OF 3.5 REMS DURING THE SECOND CALENDAR QUARTER OF 1980.
(8022 INFRACTION)

10 CFR 20.201(B) REQUIRES LICENSEES TO MAKE SURVEYS AS MAY BE NECESSARY FOR HIM TO COMPLY WITH THE REGULATIONS IN 10 CFR 20. SURVEYS ARE DEFINED IN 20.201(A) AS "AN EVALUATION OF THE RADIATION HAZARDS INCIDENT TO THE PRODUCTION, USE, RELEASE, DISPOSAL, OR PRESENCE OF RADIOACTIVE MATERIALS OR OTHER SOURCES OF RADIATION UNDER A SPECIFIC SET OF CONDITIONS. WHEN APPROPRIATE, SUCH EVALUATION INCLUDES A PHYSICAL SURVEY OF THE LOCATION OF MATERIALS AND EQUIPMENT, AND MEASUREMENTS OF LEVELS OF RADIATION OR CONCENTRATIONS OF RADIOACTIVE MATERIAL PRESENT." CONTRARY TO THE REQUIREMENTS, EVALUATIONS OF THE RADIATION DOSE RECEIVED BY AN INDIVIDUAL DURING THE SECOND CALENDAR QUARTER OF 1980 WERE NOT ADEQUATE TO ASSURE COMPLIANCE WITH 10 CFR 20.101(B) IN THAT A CONTRACTOR EMPLOYED INDIVIDUAL RECEIVED A DOSE OF 3.5 REM.
(8022 INFRACTION)

TECHNICAL SPECIFICATION SECTION 6.11 REQUIRES THAT WRITTEN PROCEDURES FOR PERSONNEL RADIATION PROTECTION SHALL BE PREPARED CONSISTENT WITH THE REQUIREMENTS OF 10 CFR 20 AND SHALL BE APPROVED, MAINTAINED AND ADHERED TO FOR ALL OPERATIONS INVOLVING PERSONNEL RADIATION EXPOSURE. RADIATION PROTECTION PROCEDURE VII 1.5, "ACCESS TO CONTROLLED AND EXCLUSION AREAS," REVISION 3, JANUARY 1979, PART III E-3 STATES: "A POCKET DOSIMETER WILL BE ISSUED AND READ BY THE EXCLUSION AREA MONITOR PRIOR TO EACH PERSON'S ENTRY INTO THE MAIN CONTROLLED AREA. THE DOSIMETER WILL BE RETURNED TO AND READ BY THE MONITOR WHEN THE PERSON LEAVES THE CONTROLLED AREA." CONTRARY TO THE REQUIREMENTS, AT NUMEROUS TIMES DURING THE PERIOD FROM APRIL 15 TO JUNE 15, 1980, INDIVIDUALS WHO WERE LEAVING THE CONTROLLED AREA AND WHO WERE NOT THE EXCLUSION AREA MONITORS, READ AND REPORTED THEIR OWN POCKET DOSIMETERS.
(8022 INFRACTION)

INSPECTION STATUS

SYSTEMS AND COMPONENT PROBLEMS:

REFUELING OUTAGE COMMENCED TWO DAYS EARLIER THAN SCHEDULED DUE TO INCREASE IN PRIMARY TO SECONDARY (S/G TUBE) LEAKAGE (PNO-V-80-24).

+ LARGE NUMBER OF TUBES DETERMINED TO NEED PLUGGING (RESULTS OF EDDY CURRENT TESTING). LICENSEE PLANS TO INSTALL SLEEVES IN TUBES IN LIEU OF PLUGGING.

FACILITY ITEMS (PLANS AND PROCEDURES):

+ STEAM GENERATOR REPAIRS.

MANAGERIAL ITEMS

NONE.

PLANT STATUS:

+ PLANT SHUTDOWN FOR STEAM GENERATOR REPAIRS.

LAST IE SITE INSPECTION DATE: 09/04-26/80+

INSPECTION REPORT NO: 50-206/80-28+

SUMMARY

INSPECTION SUMMARY

+ INSPECTION ON AUGUST 18-22, 1980 (REPORT NO. 50-206/80-23) AREAS INSPECTED: SPECIAL, UNANNOUNCED INSPECTION BY REGIONAL BASED INSPECTORS TO REVIEW THE RADIATION PROTECTION PLANNING AND PREPARATIONS ASSOCIATED WITH THE IMPENDING STEAM GENERATOR REPAIR ACTIVITY AND TO REVIEW THE CIRCUMSTANCES SURROUNDING THE SPILL OF LIQUID RADIOACTIVE WASTE REPORTED TO REGION V ON AUGUST 13, 1980. THE INSPECTION INVOLVED 70 INSPECTOR HOURS ONSITE BY THREE NRC INSPECTORS.

- RESULTS: OF THE TWO AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED. HOWEVER, BASED ON THE TENTATIVE REPAIR SCHEDULE AND THE FINDINGS OF RECENT NRC RADIATION PROTECTION INSPECTIONS, A CORPORATE MANAGEMENT MEETING WAS SCHEDULED FOR SEPTEMBER 5, 1980.

+ INSPECTION ON AUGUST 11-29, 1980 (REPORT NO. 50-206/80-25) AREAS INSPECTED: ROUTINE, RESIDENT INSPECTION OF PLANT OPERATIONS DURING LONG TERM OUTAGE, MONTHLY MAINTENANCE AND SURVEILLANCE OBSERVATIONS, AND REVIEW OF LICENSEE EVENT REPORTS. THE INSPECTION INVOLVED 60 INSPECTOR HOURS ONSITE BY ONE NRC INSPECTOR.

RESULTS: NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

+ INSPECTION ON SEPTEMBER 22-26, 1980 (REPORT NO. 50-206/80-26) REPORT BEING PREPARED; TO BE REPORTED NEXT MONTH.

+ INSPECTION ON SEPTEMBER 15-19, 1980 (REPORT NO. 50-206/80-27) REPORT BEING PREPARED; TO BE REPORTED NEXT MONTH.

+ INSPECTION ON SEPTEMBER 4-26, 1980 (REPORT NO. 50-206/80-28) REPORT BEING PREPARED; TO BE REPORTED NEXT MONTH.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

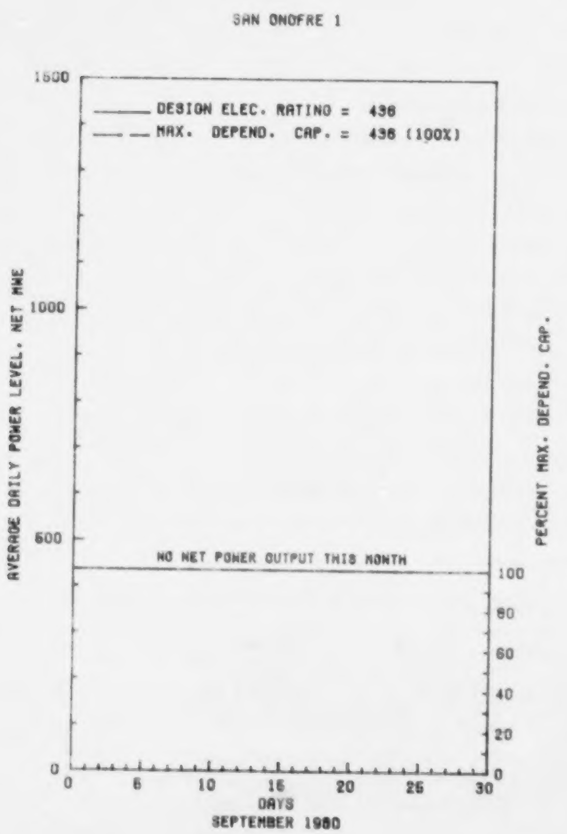
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-10	10-03-80	10-03-80	EXPOSURE IN EXCESS OF 10 CFR 20.101(A) VALUES (SPECIAL REPORT).
80-33/ 01T-0	09-02-80	09-16-80	DESIGN PROBLEM WITH SEQUENCER SYSTEM.
80-34/ 03L-0	09-01-80	09-17-80	RCS BORON DILUTION-SG SEC WATER ENTERED RCS WHEN SG TUBE REMOVED.
80-35/ 01T-0	09-18-80	10-10-80	POTENTIAL DESIGN PROBLEM.

REVIEWED BY OMPA LOEB: PE Bobe **DATE:** 11/5/80

REVIEWED BY NRR DL: S.J. Nowicki **DATE:** 11/5/80

SAN ONOFRE 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-206
2. Utility Contact: J. M. JOY (714) 492-7700
3. Licensed Thermal Power (MWt): 1347
4. Nameplate Rating (Gross MWe): 500 x 0.9 = 450
5. Design Electrical Rating (Net MWe): 436
6. Maximum Dependable Capacity (Gross MWe): 456
7. Maximum Dependable Capacity (Net MWe): 436

CAPACITY FACTORS CALCULATED WITH A WEIGHTED AVERAGE

8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	6,575.0	116,551.0
12. Hours Reactor Was Critical	.0	1,979.1	84,611.3
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	.0	1,961.6	81,114.0
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	2,552,787	104,084,261
17. Gross Electrical Energy (MWH)	0	855,000	35,521,434
18. Net Electrical Energy (MWH)	0	816,676	33,664,042
19. Unit Service Factor	.0	29.8	69.6
20. Unit Availability Factor	.0	29.8	69.6
21. Unit Capacity Factor (MDC Net)	.0	28.5	70.7
22. Unit Capacity Factor (DER Net)	.0	28.5	70.7
23. Unit Forced Outage Rate	100.0	43.7	11.8

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): STEAM GENERATOR REPAIR IN PROGRESS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: 06/81

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
53	07/12/80	[illegible]	720.0	B	4	80-01[illegible]	CC	HTEXCH	STEAM GENERATOR TUBE REPAIR CONTINUES

SUMMARY

SAN ONOFRE REMAINED OFF-LINE FOR SEPTEMBER IN A CONTINUOUS OUTAGE TO REPAIR STEAM GENERATOR.

LEGEND

[1] Type:
F-Forced
S-Scheduled

[2] Reason:
A-Equipment Failure
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error
H-Other

[3] Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Continuations
5-Load Reduction
9-Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

SEQUOYAH 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Tennessee/Hamilton 9.5 Miles NE of Chattanooga
2.	Electric Reliability Council	SERC
3.	Type of Reactor	PWR
4.	Licensed Power Level (MWT)	3411
5.	Design Electrical Rating (MWE Net)	1140
6.	Date of Initial Criticality	July 5, 1980
7.	Date of Commercial Operation	N/A
8.	Condenser Cooling Method	Natural Draft Cooling Towers
9.	Condenser Cooling Water Source	Tennessee River

Utility and Contractor Information

10.	Licensee	Tennessee Valley Authority
11.	Corporate Address	500A Chestnut Street Tower II Chattanooga, TN 37401
12.	Corporate Contact	H. G. Parris, Manager of Power
13.	Architect/Engineer	TVA
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	TVA

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. Lewis
18.	IE Section Chief	H. C. Dance
19.	IE Resident Inspector	C. Julian
20.	Licensing Project Manager	C. R. Stahle
21.	Docket Number	50-327
22.	License Number/Date of Issue	DPR-77, 9/17/80
23.	Public Document Room	Chattanooga - Hamilton Bicentennial Library 1001 Broad Street Chattanooga, Tennessee 37402

INSPECTION STATUS

ENFORCEMENT STATUS

AS REQUIRED BY SECTION 6.8.1 OF APPENDIX A TO OPERATING LICENSE DPR-77, CRITERION XIV OF 10 CFR 50, APPENDIX B AND AI-9, MEASURES SHALL BE ESTABLISHED FOR INDICATING THE OPERATING STATUS OF STRUCTURES, SYSTEMS AND COMPONENTS OF THE NUCLEAR POWER PLANT. CONTRARY TO THE ABOVE, ON JUNE 25, 1980 THE ICE CONDENSER INLET DOORS WERE BLOCKED SHUT IN ORDER TO ALLOW COLD AIR TO BUILD UP BEHIND THE DOORS TO HOLD THEM SHUT PROPERLY. THIS TEMPORARY ALTERATION OF A CRITICAL SYSTEM, DESIGNATED AS OPERABLE, WAS DONE WITHOUT THE USE OF A TEMPORARY ALTERATION CONTROL FORM OR TEMPORARY ALTERATION ORDER.
(8025 DEFICIENCY)

AS REQUIRED BY 10 CFR 73.55(D)(7), "THE LICENSEE SHALL POSITIVELY CONTROL ALL POINTS OF PERSONNEL ACCESS INTO VITAL AREAS...." IN ADDITION, SECTION 5.2.1 OF THE SEQUOYAH PHYSICAL SECURITY PLAN (PSP) STATES THAT "ALL VITAL AREA BARRIERS WILL BE RESISTANT TO PENETRATION. THE PENETRATION RESISTANCE OF THE BARRIERS WILL NOT BE DIMINISHED BY DOORS, WINDOWS AND OTHER OPENINGS IN THE BARRIERS." CONTRARY TO THE ABOVE, (A) ON JUNE 23, 1980 A VITAL AREA DOOR WAS REMOVED FROM ITS HINGES FOR MAINTENANCE WITHOUT ADEQUATE COMPENSATORY MEASURES BEING PROVIDED TO CONTROL ACCESS INTO THE VITAL AREA. (B) ON JUNE, 1980 A PROTECTIVE GRATING WAS REMOVED FROM A VITAL AREA BARRIER OPENING TO PROVIDE ACCESS TO THE AREA FOR MAINTENANCE PERSONNEL WITHOUT ADEQUATE COMPENSATORY MEASURES BEING PROVIDED TO CONTROL ACCESS INTO THE VITAL AREA. THIS IS AN INFRACTION. A SIMILAR ITEM WAS BROUGHT TO YOUR ATTENTION IN OUR LETTERS DATED JUNE 2, 1980 AND JUNE 26, 1980.
(8025 INFRACTION)

FAILURE TO FOLLOW PROCEDURE SQA-44 IN THAT, DURING PERFORMANCE OF TEST SU-7.6 (PSEUDO EJECTION OF ROD) INACCURATE RESULTS WERE RECOGNIZED BUT NO TEST DEFICIENCY WAS DOCUMENTED.
(8031 INFRACTION)

CONTRARY TO THE REQUIREMENTS OF THE PHYSICAL SECURITY PLAN, PROCEDURE PHYSI-13, TWO INDIVIDUALS WERE ASSIGNED TO THE OPERATIONS FIRE BRIGADE ON AUGUST 15, 18 AND 19 WHO HAD NOT RECEIVED REQUALIFICATION TRAINING SINCE JANUARY 1979.
(8034 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE

MANAGERIAL ITEMS:

NONE

PLANT STATUS:

FULL POWER LICENSE ISSUED ON 9/17/80.

+ LAST IE SITE INSPECTION DATE: SEPTEMBER 1-30, 1980

+ INSPECTION REPORT NO: 50-327/80-38

SUMMARY

INSPECTION SUMMARY

+ INSPECTION AUGUST 11-15 (80-27): INCLUDED REVIEW OF SECURITY PLAN; SECURITY PROGRAM AUDIT; SECURITY ORGANIZATION; PHYSICAL BARRIERS (PROTECTED AND VITAL AREAS); LIGHTING; ACCESS CONTROLS (IDENTIFICATION, AUTHORIZATION AND BADGING, SEARCH, ESCORTING, AND VITAL AREAS); ALARM STATIONS; DETECTION AIDS; ASSESSMENT AIDS; COMMUNICATIONS; TESTING AND MAINTENANCE; COMPENSATORY MEASURES; POWER SUPPLY; RESPONSE; LOCKS, KEYS AND COMBINATIONS, AND RECORDS AND REPORTS. THE INSPECTION INVOLVED 80 INSPECTOR-HOURS ON SITE BY TWO NRC INSPECTORS. FOURTEEN INSPECTION HOURS WERE ACCOMPLISHED DURING OFFSHIFT PERIODS. THE LICENSEE WAS FOUND TO BE IN COMPLIANCE WITH NRC REQUIREMENTS WITHIN THE 16 AREAS EXAMINED DURING THE INSPECTION EXCEPT FOR THE FOLLOWING 3 ITEMS: (A) PHYSICAL BARRIERS - VITAL AREA BARRIER RESISTANCE TO PENETRATION DIMINISHED BY INADEQUATE VENT COVERING; (B) DETECTION AIDS - PERIM - ALERT II ALARM SYSTEM NOT REDUNDANT; (C) DETECTION AIDS - PERIM - ALERT II ALARM SYSTEM TERMINAL BOXES NOT EQUIPPED WITH TAMPER ALARMS.

INSPECTION JULY 6 - JULY 31 (80-29): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 215 INSPECTOR-HOURS ON SITE IN THE AREAS OF OPERATIONAL SAFETY VERIFICATION, START-UP TEST WITNESSING, SPECIAL TEST WITNESSING, VERIFICATION OF LICENSE CONDITIONS, LICENSEE EVENT REPORT REVIEW, IE BULLETIN REVIEW, INDEPENDENT INSPECTION EFFORT AND FOLLOWUP ON PLANT INCIDENTS. OF THE EIGHT AREAS INSPECTED NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION JULY 29 (80-30): INSPECTED CHEM-NUCLEAR WASTE DISPOSAL SITE, BARNWELL, SOUTH CAROLINA FOR UNANNOUNCED INSPECTIONS OF LICENSEE-SHIPPERS. EACH LICENSEE-SHIPPER VEHICLE WAS INSPECTED FOR COMPLIANCE WITH DEPARTMENT OF TRANSPORTATION (DOT) AND NUCLEAR REGULATORY COMMISSION (NRC) REGULATIONS AS FOLLOWS: (1) SHIPPING PAPER REQUIREMENTS; (2) DOT PLACARDING REQUIREMENTS; (3) RADIATION LEVELS; (4) REMOVABLE CONTAMINATION; (5) DOT MARKING AND LABELING REQUIREMENTS FOR PACKAGES; (6) DOT AND NRC REQUIREMENTS FOR EXTERNAL PACKAGE FEATURES, AND (7) PROHIBITED ARTICLES OR CONTENTS. OF THE SEVEN AREAS INSPECTED INVOLVING SHIPMENTS, ONE ITEM OF NONCOMPLIANCE WAS IDENTIFIED.

INSPECTION JULY 22-25 (80-31): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 30 INSPECTOR-HOURS ON SITE IN THE AREAS OF REVIEW OF RESULTS OF LOW POWER PHYSICS TESTING AND ON FOLLOWUP OF PREVIOUS LICENSEE EVENT REPORT. OF THE 2 AREAS INSPECTED, ONE ITEM OF NONCOMPLIANCE WAS FOUND. (INFRACTION - FAILURE TO DOCUMENT A TEST DEFICIENCY).

INSPECTION AUGUST 11-13 (80-32): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 14 INSPECTOR-HOURS ON SITE IN THE AREAS OF FOLLOWUP ON ITEMS OF NONCOMPLIANCE RELATIVE TO 10 CFR PART 21 PROCEDURES AND PROCEDURE IMPLEMENTATION. THE PREVIOUSLY IDENTIFIED ITEMS OF NONCOMPLIANCE AND INSPECTOR FOLLOWUP ITEM REMAIN OPEN.

INSPECTION AUGUST 20-22 (80-34): THIS ROUTINE ANNOUNCED INSPECTION INVOLVED 79 INSPECTOR-HOURS ON SITE IN THE AREAS OF: EMERGENCY FACILITIES, EQUIPMENT AND PROCEDURES; MEANS FOR DETERMINING A RELEASE; EMERGENCY TRAINING FOR LICENSEE EMPLOYEES AND OFFSITE GROUPS; FIRE BRIGADE ORGANIZATION AND TRAINING; EMERGENCY ORGANIZATION; FOLLOWUP ON IE BULLETINS; EMERGENCY TESTS AND DRILLS; AND EMERGENCY PLAN SAFETY EVALUATION REPORT. OF THE EIGHT AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN SEVEN AREAS; ONE APPARENT ITEM OF NONCOMPLIANCE WAS FOUND IN ONE AREA (DEFICIENCY - FAILURE TO IMPLEMENT PROCEDURES).

INSPECTION AUGUST 14-15 (80-35): THIS SPECIAL ANNOUNCED INSPECTION INVOLVED 24 INSPECTOR AND INVESTIGATOR HOURS AT TVA OFFICES AT KNOXVILLE AND CHATTANOOGA, TN INQUIRING INTO TVA'S WITHHOLDING INFORMATION FROM THE ADVISORY COMMITTEE ON REACTOR SAFEGUARDS (ACRS) DURING MEETINGS HELD JULY 9 AND 11, 1980. NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

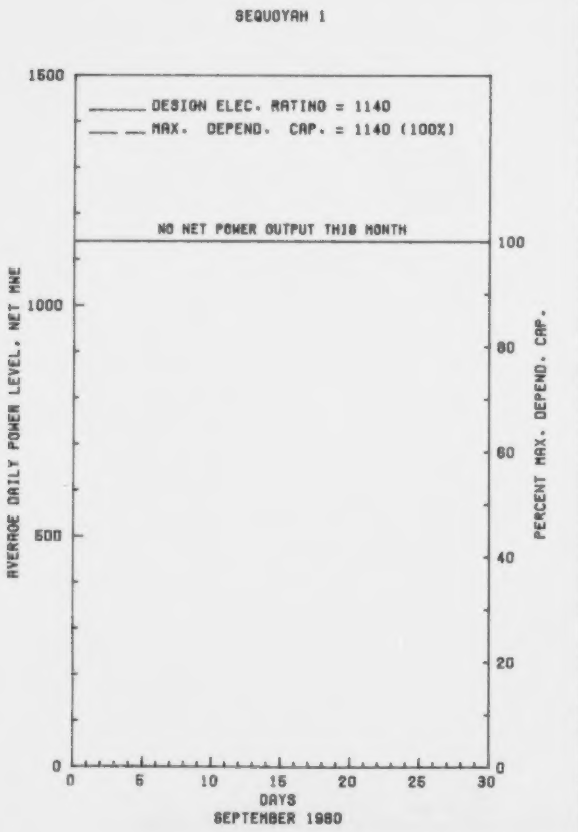
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-012/ 03L-0	05/01/80	06/18/80	SPENT FUEL GATES.
80-079/ 03L-0	07/16/80	08/19/80	ICE BED TEMPERATURES EXCEED LIMIT DURING TEST.
80-126/ 03L-0	06/27/80	08/13/80	LOWER CONTAINMENT PURGE EXHAUST VALVE OPEN CONTRARY TO LCO.
80-127/ 03L-0	06/30/80	08/13/80	FAILED TO PERFORM SURVEILLANCE ON REACTOR COOLANT SYSTEM LEAK RATE.
80-131/ 03L-0	06/26/80	08/13/80	FAILED TO MEET SURVEILLANCE ON REACTOR COOLANT SYSTEM UNIDENTIFIED LEAK RATE.
80-133/ 03L-0	07/18/80	08/19/80	ONE HYDROGEN ANALYZER IN CONTAINMENT BECAME INOPERABLE DUE TO OVERHEATED BREAKER.
80-137/ 03L-0	07/28/80	08/21/80	WASTE GAS EFFLUENT MONITOR INOPERABLE.
80-139/ 03L-0	08/07/80	09/04/80	COMPONENT COOLING WATER PUMP POWER SUPPLY TRIPPED.
80-143/ 03L-0	07/31/80	09/03/80	THREE OF FOUR DIESEL GENERATORS FAILED TO START IN RESPONSE TO PREOP TEST.
80-145/ 03L-0	08/07/80	09/04/80	DIESEL GENERATOR 1B-B BECAME INOPERABLE DUE TO BATTERY FAILURE.
80-146/ 03L-0	06/17/80	09/08/80	FAILED TO COMPLETE ONE STEP OF VENTING INSTRUCTIONS DURING SURVEILLANCE ON SAFETY INJECTION PUMPS.
80-147/ 03L-0	08/07/80	09/04/80	FOUND ADDITIONAL HANGER ON CONTAINMENT SPRAY PIPING.
80-149/ 03L-0	08/07/80	09/04/80	TURBINE AUXILIARY FEED PUMP INOPERABLE DUE TO OVERHEATING OF CONTROLLER.
80-152/ 03L-0	08/12/80	09/11/80	DIESEL GENERATOR BATTERY CELLS DISCHARGED.
80-155/ 03L-0	08/13/80	09/11/80	REMOTE SHUTDOWN PRESSURIZER RELIEF TANK LEVEL INDICATION BECAME INOPERABLE.
80-060/ 03L-0	05/21/80	08/28/80	LIQUID EFFLUENT RADIATION MONITOR INOPERABLE DUE TO ELECTRICAL INTERFERENCE.

REVIEWED BY OMPA LOEB: Philip A. Ross **DATE:** 11/05/80

REVIEWED BY NRR DL: Carl R Stahle **DATE:** 11/6/80

SEQUOYAH 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-327
2. Utility Contact: DAVID DUPREE (615) 842-0295
3. Licensed Thermal Power (MWt): 3411
4. Nameplate Rating (Gross MWe): 1220.5
5. Design Electrical Rating (Net MWe): 1140
6. Maximum Dependable Capacity (Gross MWe): 1185
7. Maximum Dependable Capacity (Net MWe): 1140
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	1,440.0	1,440.0
12. Hours Reactor Was Critical	19.2	542.0	542.0
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	.0	18.4	18.4
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	10,517	39,360	39,360
17. Gross Electrical Energy (MWH)			
18. Net Electrical Energy (MWH)			
19. Unit Service Factor			
20. Unit Availability Factor			
21. Unit Capacity Factor (MDC Net)			
22. Unit Capacity Factor (DER Net)			
23. Unit Forced Outage Rate			

NOT IN COMMERCIAL OPERATION

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause and Corrective Action to Prevent Recurrence
NONE									

SUMMARY

SEQUOYAH 1 RECEIVED FULL-POWER LICENSE ON 09/17/80 AND WENT INTO MODE 4 ON 09/26/80. UNIT WAS CRITICAL AT 0450 ON 09/30/80 AND ENTERED MODE 2.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & Licensee Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source

ST. LUCIE 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Florida/St. Lucie 12 miles SE of Ft. Pierce
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2560
5.	Design Electrical Rating (MWE Net)	802
6.	Date of Initial Criticality	April 22, 1976
7.	Date of Commercial Operation	December 21, 1976
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Atlantic Ocean

Utility & Contractor Information

10.	Licensee	Florida Power & Light Co.
11.	Corporate Address	9250 W. Flagler Street P. O. Box 529100 Miami, Florida 33152
12.	Corporate Contact	R. E. Uhrig, V.P.
13.	Architect/Engineer	Ebasco
14.	Nuclear Steam Supply System	Combustion Engineering
15.	Constructor	Ebasco

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. LEWIS
18.	IE Section Chief	R. A. Martin
19.	IE Resident Inspector	S. A. ELROD
20.	Licensing Project Manager	C. Nelson
21.	Docket Number	50-335
22.	License Number/Date of Issue	DPR-67, March 1, 1976
23.	Public Document Room	Indian River Junior College Library 3209 Virginia Avenue Ft. Pierce, Florida 33450

INSPECTION STATUS

ENFORCEMENT STATUS

AS REQUIRED BY 10CFR50.72(A)(3), LICENSEES ARE REQUIRED TO NOTIFY THE NRC OPERATIONS CENTER AS SOON AS POSSIBLE AND IN ALL CASES WITHIN ONE HOUR OF ANY EVENT THAT RESULTS IN A NUCLEAR POWER PLANT NOT BEING IN A CONTROLLED OR EXPECTED CONDITION WHILE OPERATING OR SHUT DOWN. CONTRARY TO THE ABOVE, ON JUNE 11, 1980, DURING A NATURAL CIRCULATION COOLDOWN: (A) ST. LUCIE UNIT NO. 1 FORMED A STEAM BUBBLE IN THE REACTOR HEAD AREA; (B) INDICATIONS OF THIS PHENOMENON BEGAN TO BE SEEN ABOUT 6:3[illegible] A.M. AND CONTINUED THROUGHOUT THE MORNING; (C) THE PLANT OPERATORS RECOGNIZED THAT THE PLANT WAS NOT IN AN EXPECTED CONDITION BY ABOUT 8:00 A.M. AND (D) THE NRC OPERATIONS CENTER WAS NOT NOTIFIED UNTIL AFTER 11:00 A.M.
(8017 INFRACTION)

CONTRARY TO 10 CFR 50, APPENDIX B, CRITERION V, ON MAY 15, 1980 REVISION 17 OF PROCEDURE OP 0030122 WAS USED FOR REACTOR STARTUP. REVISION 18 OF THAT PROCEDURE HAD BEEN ISSUED ON FEBRUARY 28, 1980 AND HAD BEEN IN USE SINCE MAY 7, 1980.
(8025 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

NORMAL OPERATION.

+ LAST IE SITE INSPECTION DATE: OCTOBER 14-17, 1980

+ INSPECTION REPORT NO: 50-335/80-32

SUMMARY

INSPECTION SUMMARY

+ INSPECTION AUGUST 5-10 (80-24): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 42 INSPECTOR-HOURS ON SITE IN THE AREAS OF OPERATIONAL PRACTICES AND FOLLOWUP ON EVENTS OCCURRING WHILE ON SITE. OF THE TWO AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION AUGUST 19-22 (80-29): A REVIEW OF LICENSEE ACTION ON PREVIOUSLY IDENTIFIED ITEMS ON NONCOMPLIANCE IN THE AREAS OF SECURITY ORGANIZATION, PHYSICAL BARRIERS (VITAL AREA), ACCESS CONTROL (SEARCH), AND ACCESS CONTROL (VITAL AREAS). ALSO INSPECTED WERE THE SECURITY PLAN, PHYSICAL BARRIERS (PROTECTED AREA) COMPENSATORY MEASURES, POWER SUPPLY, AND RESPONSE. THE INSPECTION INVOLVED 31 INSPECTOR-HOURS ON SITE BY ONE NRC INSPECTOR. THE INSPECTION WAS BEGUN DURING AN OFFSHIFT PERIOD. NINE INSPECTION HOURS WERE ACCOMPLISHED DURING OFFSHIFT PERIODS. THE LICENSEE WAS FOUND TO BE IN COMPLIANCE WITH NRC REQUIREMENTS WITHIN THE 9 AREAS EXAMINED DURING THE INSPECTION EXCEPT FOR THE FOLLOWING 3 ITEMS: (1) ACCESS CONTROL (SEARCH) - INADEQUATE PROCEDURES FOR SEARCHING PACKAGES; (2) ACCESS CONTROL (VITAL AREAS) - FAILURE TO POSITIVELY CONTROL ALL POINTS OF PERSONNEL AND VEHICLE ACCESS INTO VITAL AREAS; AND (3) ACCESS CONTROL (VITAL AREAS) - FAILURE TO HAVE AN ALARM ON A VITAL AREA PORTAL.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

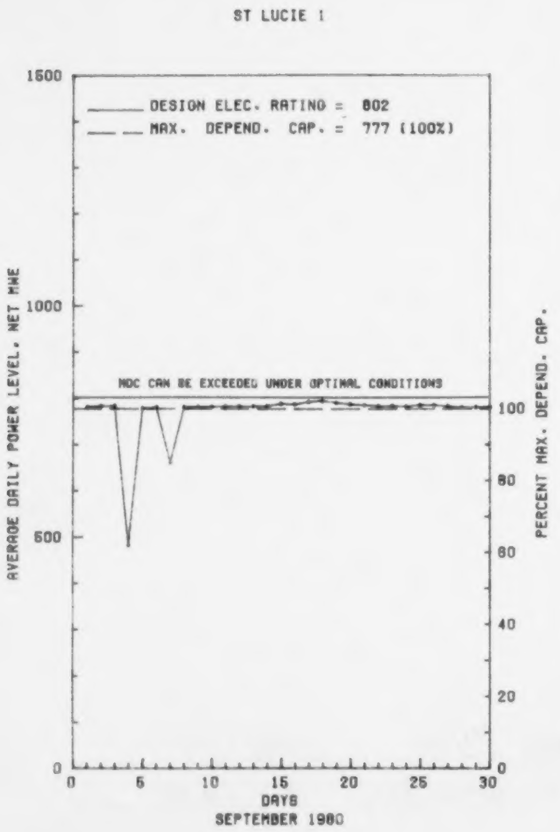
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-037/03L-0	08/12/80	09/11/80	RADIATION CHANNEL "MD" WAS INOPERABLE DUE TO FAILED CAPACITORS.
80-038/03L-0	08/13/80	09/12/80	CONTROL ELEMENT DRIVE MOTOR FAILED DUE TO A TIMING MODULE PROBLEM.

REVIEWED BY OMPA LOEB: M.R. Beebe **DATE:** 11/04/80

REVIEWED BY NRR DL: **DATE:** 11-5-80

ST. LUCIE 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-335
2. Utility Contact: V. T. Chilson (305) 552-3824
3. Licensed Thermal Power (MWt): 2560
4. Nameplate Rating (Gross MWe): 1000 x 0.85 = 850
5. Design Electrical Rating (Net MWe): 802
6. Maximum Dependable Capacity (Gross MWe): 822
7. Maximum Dependable Capacity (Net MWe): 777
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	6,575.0	33,119.0
12. Hours Reactor Was Critical	717.3	4,717.5	26,227.4
13. Reactor Reserve Shutdown Hours	.0	.0	129.4
14. Hours Generator On-Line	716.4	4,606.2	25,436.3
15. Unit Reserve Shutdown Hours	.0	7.3	39.3
16. Gross Thermal Energy (MWH)	1,810,963	11,386,317	61,645,883
17. Gross Electrical Energy (MWH)	585,790	3,687,280	19,981,780
18. Net Electrical Energy (MWH)	553,858	3,468,894	18,795,790
19. Unit Service Factor	99.5	70.1	76.8
20. Unit Availability Factor	99.5	73.2	76.9
21. Unit Capacity Factor (MDC Net)	99.0	67.9	73.0
22. Unit Capacity Factor (DER Net)	95.9	65.8	70.8
23. Unit Forced Outage Rate	.5	9.6	6.2

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
15	09/04/80	S	.0	B	5		RB	INSTRU	LOAD REDUCTION TO APPROXIMATELY 78% R.P. TO CONTINUE MODIFICATION OF 15 VOLT POWER SUPPLY TO CEA DRIVE MECHANISM CORE POWER PROGRAMMER.
16	09/04/80	F	3.6	A	2	335-80-50	RB	INSTRU	REACTOR WAS MANUALLY TRIPPED WHEN SECOND CEA DROPPED BEFORE CEA NO. 44 COULD BE REALIGNED WITH ITS BANK. CORRECTIVE ACTIONS INCLUDED MODIFICATION OF 15 VOLT POWER SUPPLY TO CEA'S.

SUMMARY

ST. LUCIE OPERATED AT OR NEAR FULL POWER WITH THE EXCEPTION OF A MODIFICATION REDUCTION ON THE 4TH AND ALSO A REPAIR OUTAGE TO 15 VOLT CEA POWER SUPPLY ON THE 4TH.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source

SURRY 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Virginia/Surry 17 miles NW of Newport News
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2441
5.	Design Electrical Rating (MWE Net)	822
6.	Date of Initial Criticality	July 1, 1972
7.	Date of Commercial Operation	December 22, 1972
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	James River

Utility & Contractor Information

10.	Licensee	Virginia Electric and Power Company
11.	Corporate Address	P. O. Box 26666 Richmond, Virginia 23261
12.	Corporate Contact	J. H. Ferguson Exec. V.P. - Power
13.	Architect/Engineer	Stone & Webster
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R.C. Lewis
18.	IE Section Chief	P. J. Kellogg
19.	IE Resident Inspector	D. Burke (804) 357-2101
20.	Licensing Project Manager	D. Neighbors
21.	Docket Number	50-280
22.	License Number/Date of Issue	DPR-32, 5/26/72
23.	Public Document Room	Swem Library College of William and Mary Williamsburg, Virginia 23185

INSPECTION STATUS

ENFORCEMENT STATUS

AS REQUIRED BY CRITERION V OF APPENDIX B TO 10 CFR 50, AS IMPLEMENTED BY VIRGINIA ELECTRIC AND POWER COMPANY TOPICAL REPORT VEP-1-3A, "QUALITY ASSURANCE PROGRAM - OPERATIONS PHASE," SECTION 17.2.5, "ACTIVITIES AFFECTING QUALITY SHALL BE PRESCRIBED BY DOCUMENTED INSTRUCTIONS, PROCEDURES, ...". IE BULLETIN 79-14, "SEISMIC ANALYSIS FOR AS-BUILT SAFETY RELATED PIPING SYSTEMS," SPECIFIES THAT THE LICENSEE SHOULD ASSURE THAT HE IS PROMPTLY NOTIFIED WHEN THE ARCHITECT/ENGINEER IDENTIFIES A NONCONFORMANCE AND THAT THE SIGNIFICANCE OF NONCONFORMANCES WITH RESPECT TO SYSTEM OPERABILITY BE EVALUATED BY ENGINEERING JUDGEMENT WITHIN TWO DAYS OF IDENTIFICATION AND BY ANALYSIS WITHIN 30 DAYS OF IDENTIFICATION. CONTRARY TO THE ABOVE, PROCEDURES ISSUED TO ACCOMPLISH IE BULLETIN 79-14 DO NOT PROPERLY ADDRESS THE TIMELY EVALUATION AND ANALYSIS OF IDENTIFIED NONCONFORMANCES OR THE PROMPT NOTIFICATION OF VEPCO OF THESE ITEMS. STRESS PROBLEM 3033 ILLUSTRATES THE RESULTS OF THESE INADEQUATE PROCEDURES IN THAT SIGNIFICANT OVERSTRESSES IDENTIFIED BY THE INITIAL ANALYSIS ON DECEMBER 1, 1979 WERE NOT EXAMINED BY AN INDEPENDENT REVIEWER UNTIL FEBRUARY 1, 1980. ON THAT DATE VEPCO SHUT THE UNIT DOWN DUE TO CONTAINMENT INTEGRITY QUESTIONS RESULTING FROM THIS PROBLEM. (8031 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

STEAM GENERATOR TUBE LEAKS.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

+ UNIT SHUTDOWN FOR S/G REPAIR.

+ LAST IE SITE INSPECTION DATE: AUGUST 22, 1980

INSPECTION REPORT NO: 50-280/80-33

SUMMARY

INSPECTION SUMMARY

+ INSPECTION AUGUST 5-6 (80-31): THIS SPECIAL, ANNOUNCED INSPECTION INVOLVED 10 INSPECTOR-HOURS ON SITE IN THE AREAS OF LICENSEE ACTIONS ON PREVIOUS INSPECTION FINDINGS. ONE ITEM OF NONCOMPLIANCE WAS IDENTIFIED (INFRACTION - IMPROPER PIPE SUPPORT INSTALLATION AND INSPECTION).

INSPECTION AUGUST 20-22 (80-32): THIS ROUTINE UNANNOUNCED INSPECTION INVOLVED 9 INSPECTOR-HOURS ON SITE IN THE AREAS OF IE BULLETIN 79-02, "PIPE SUPPORT BASE PLATE DESIGNS USING CONCRETE EXPANSION ANCHOR BOLTS" AND IE BULLETIN 79-14, "SEISMIC ANALYSIS FOR AS-BUILT SAFETY RELATED PIPING SYSTEMS." OF THE TWO AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN ONE AREA; ONE ITEM OF NONCOMPLIANCE WAS FOUND IN ONE AREA (INFRACTION - IMPROPER INSTALLATION AND INSPECTION OF PIPE SUPPORT).

INSPECTION AUGUST 20-22 (80-33): INCLUDED REVIEW OF ACCESS CONTROL (AUTHORIZATION); LIGHTING; POWER SUPPLY AND FOLLOWUP ON PREVIOUS ENFORCEMENT MATTERS. THE INSPECTION INVOLVED 7 INSPECTOR-HOURS ON SITE BY ONE NRC INSPECTOR. THE INSPECTION WAS BEGUN DURING AN OFF-SHIFT PERIOD; TWO INSPECTION HOURS WERE ACCOMPLISHED DURING OFFSHIFT PERIODS. WITHIN THE THREE AREAS INSPECTED, THERE WERE NO ITEMS OF NONCOMPLIANCE IDENTIFIED.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

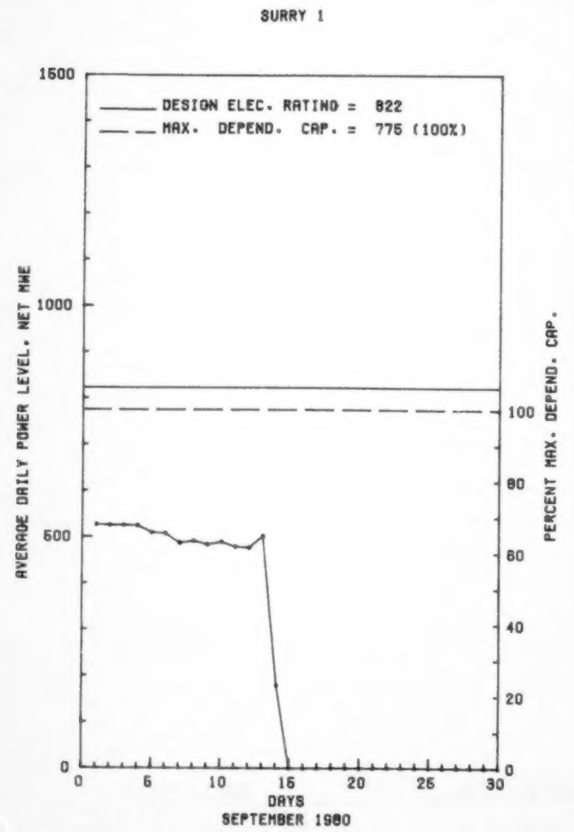
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE.			

REVIEWED BY OMPA LOEB: PEBobe **DATE:** 11/5/80

REVIEWED BY NRR DL: D. Neighbors **DATE:** 11/6/80

SURRY 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-280
2. Utility Contact: SUE DUNN (804) 357-3184
3. Licensed Thermal Power (MWt): 2441
4. Nameplate Rating (Gross MWe): 942 x 0.9 = 848
5. Design Electrical Rating (Net MWe): 822
6. Maximum Dependable Capacity (Gross MWe): 811
7. Maximum Dependable Capacity (Net MWe): 775
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	6,575.0	68,159.0
12. Hours Reactor Was Critical	324.2	3,786.9	42,538.5
13. Reactor Reserve Shutdown Hours	.0	182.6	3,731.5
14. Hours Generator On-Line	323.7	3,763.8	41,668.8
15. Unit Reserve Shutdown Hours	.0	183.6	3,736.2
16. Gross Thermal Energy (MWH)	616,659	8,674,529	96,389,401
17. Gross Electrical Energy (MWH)	173,010	2,621,245	31,301,743
18. Net Electrical Energy (MWH)	160,829	2,473,025	29,699,924
19. Unit Service Factor	45.0	57.2	61.1
20. Unit Availability Factor	45.0	60.0	66.6
21. Unit Capacity Factor (MDC Net)	28.8	48.5	56.2
22. Unit Capacity Factor (DER Net)	27.2	45.8	53.0
23. Unit Forced Outage Rate	.0	39.1	26.4

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 09/01/81

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-13	09/14/80	S	396.3	A	1				NORMAL SHUTDOWN FOR STEAM GENERATOR REPAIR OUTAGE. PRIMARY TO SECONDARY LEAKAGE IN "B" STEAM GENERATOR WAS .283 GPM AT TIME OF SHUTDOWN. ALL THREE STEAM GENERATOR LOWER SHELLS AND TUBE BUNDLES WILL BE REPLACED.

SUMMARY

SURRY I OPERATED NORMALLY DURING SEPTEMBER UNTIL THE 14TH, WHEN IT ENTERED AN EXTENDED OUTAGE FOR REPLACEMENT OF STEAM GENERATOR LOWER SHELLS AND TUBE BUNDLES.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

SURRY 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Virginia/Surry 17 miles NW of Newport News
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2441
5.	Design Electrical Rating (MWE Net)	822
6.	Date of Initial Criticality	March 7, 1973
7.	Date of Commercial Operation	May 1, 1973
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	James River

Utility & Contractor Information

10.	Licensee	Virginia Electric and Power Company
11.	Corporate Address	P. O. Box 26666 Richmond, Virginia 23261
12.	Corporate Contact	J. H. Ferguson Exec. V.P. - Power
13.	Architect/Engineer	Stone & Webster
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R.C. Lewis
18.	IE Section Chief	P. J. Kellog
19.	IE Resident Inspector	D. Burke (804) 357-2101
20.	Licensing Project Manager	D. Neighbors
21.	Docket Number	50-281
22.	License Number/Date of Issue	DPR-37, 1/29/73
23.	Public Document Room	Swem Library College of William and Mary Williamsburg, Virginia 23185

INSPECTION STATUS

ENFORCEMENT STATUS

AS REQUIRED BY CRITERION X OF APPENDIX B TO 10CFR50, AS IMPLEMENTED BY VIRGINIA ELECTRIC AND POWER COMPANY TOPICAL REPORT VEP-1-3A, "QUALITY ASSURANCE PROGRAM - OPERATIONS PHASE", SECTION 17.2.10, "A PROGRAM FOR INSPECTION OF ACTIVITIES AFFECTING QUALITY SHALL BE ESTABLISHED AND EXECUTED...TO VERIFY CONFORMANCE WITH THE DOCUMENTED INSTRUCTIONS, PROCEDURES AND DRAWINGS...". CONTRARY TO THE ABOVE, THREE COTTER PINS HAD NOT BEEN SPREAD AND ONE COTTER PIN WAS MISSING FROM THE STRUT CLEVIS PINS OF RECENTLY INSTALLED CONTAINMENT SPRAY SYSTEM PIPE SUPPORT 11548-BZ-4A-8. IN ADDITION, SPALLED CONCRETE INDICATED THAT A TENSION FAILURE (CONCRETE) HAD OCCURRED IN ONE CONCRETE EXPANSION ANCHOR INSTALLATION ON THIS SUPPORT. THESE CONDITIONS WERE OBSERVED ON AUGUST 21, 1980.
(8035 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

SYSTEM MODIFICATIONS AND REANALYSIS FOR SEISMIC SUPPORTS ARE CONTINUING.

FACILITY ITEMS (PLANS AND PROCEDURES):

+ NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

UNIT IS CURRENTLY AT 100% POWER AFTER CONDUCTING LOW POWER TESTING. THE UNIT HAS BEEN REFUELED.

+ LAST IE SITE INSPECTION DATE: AUGUST 22, 1980

+ INSPECTION REPORT NO: 50-281/80-36

SUMMARY

INSPECTION SUMMARY

+ INSPECTION AUGUST 20-22 (80-35): THIS ROUTINE UNANNOUNCED INSPECTION INVOLVED 8 INSPECTOR-HOURS ON SITE IN THE AREAS OF IE BULLETIN 79-02, "PIPE SUPPORT BASE PLATE DESIGNS USING CONCRETE EXPANSION ANCHOR BOLTS" AND IE BULLETIN 79-14, "SEISMIC ANALYSIS FOR AS-BUILT SAFETY RELATED PIPING SYSTEMS." OF THE TWO AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN ONE AREA; ONE ITEM OF NONCOMPLIANCE WAS FOUND IN ONE AREA (INFRACTION - IMPROPER INSTALLATION AND INSPECTION OF PIPE SUPPORT 11548-BZ-4A-8).

INSPECTION AUGUST 20-22 (80-36): INCLUDED REVIEW OF ACCESS CONTROL (AUTHORIZATION); LIGHTING; POWER SUPPLY AND FOLLOWUP ON PREVIOUS ENFORCEMENT MATTERS. THE INSPECTION INVOLVED 7 INSPECTOR-HOURS ON SITE BY ONE NRC INSPECTOR. THE INSPECTION WAS BEGUN DURING AN OFFSHIFT PERIOD; TWO INSPECTION HOURS WERE ACCOMPLISHED DURING OFFSHIFT PERIODS. WITHIN THE THREE AREAS INSPECTED, THERE WERE NO ITEMS OF NONCOMPLIANCE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

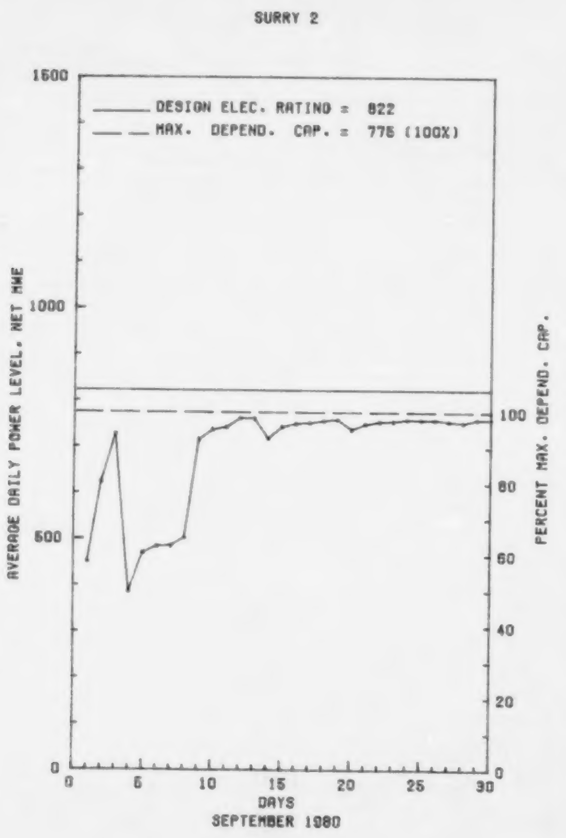
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-007/	07/18/80	07/31/80	WESTINGHOUSE REANALYSIS LOCA-ECCS INDICATES ERROR IN FUEL CLAD BURST MODEL.

REVIEWED BY OMPA LOEB: DATE: 11/5/80

REVIEWED BY NRR DL: Don Neighbors DATE: 11/6/80

SURRY 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-281
2. Utility Contact: SUE DUNN (804) 357-3184
3. Licensed Thermal Power (MWt): 2441
4. Nameplate Rating (Gross MWe): 942 x 0.9 = 848
5. Design Electrical Rating (Net MWe): 822
6. Maximum Dependable Capacity (Gross MWe): 811
7. Maximum Dependable Capacity (Net MWe): 775
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	6,575.0	65,039.0
12. Hours Reactor Was Critical	717.6	1,083.7	35,583.6
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	713.5	945.7	34,961.9
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,618,763	,919,687	81,113,770
17. Gross Electrical Energy (MWH)	524,888	611,060	26,479,904
18. Net Electrical Energy (MWH)	495,153	572,638	25,109,243
19. Unit Service Factor	99.1	14.4	53.7
20. Unit Availability Factor	99.1	14.4	53.7
21. Unit Capacity Factor (MDC Net)	88.7	11.2	49.8
22. Unit Capacity Factor (DER Net)	83.7	10.6	47.0
23. Unit Forced Outage Rate	.9	8.4	20.7

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): TMI MODS - 12/80 - (1 WEEK)
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

II

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-16	09/01/80	F	2.9	G	3				REACTOR TRIP CAUSED BY "A" S/G LO-LO LEVEL AS A RESULT OF "B" MFP TRIPPING. THE "B" MFP TRIPPED AS A RESULT OF TECHNICIANS WORKING ON THE RECIRCULATION FLOW CIRCUITRY. THE MOTOR PROTECTION LOGIC CIRCUIT SENSED A LOSS OF FLOW AND TRIPPED THE MOTOR BREAKER. REPAIRS TO THE RECIRCULATION FLOW SWITCH WERE COMPLETED AND THE "B" MFP WAS RETURNED TO SERVICE.
80-17	09/04/80	F	3.6	A	3				REACTOR TRIP CAUSED BY "A" S/G LO-LO LEVEL AS A RESULT OF THE "A" MFRV FAILING SHUT. THE "A" MFRV FAILED SHUT DUE TO A BREAK IN THE INSTRUMENT AIR LINE SUPPLYING IT. THE AIR LINE WAS REPLACED.
80-18	09/05/80	F	.0	A	5				POWER RESTRICTED TO 70% AT 0850, 09/05/80 FOR "B" MAIN FEED PUMP SEAL REPAIRS. "B" MFP RETURNED TO SERVICE AT 1856, 09/08/80.

SUMMARY

SURRY II OPERATED AT NEAR FULL POWER FOR MOST OF SEPTEMBER WITH TWO OUTAGES AND ONE REDUCTION FOR REPAIRS AND OPERATIONAL ERROR.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

THREE MILE ISLAND 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Pennsylvania/Dauphin 10 miles SE of Harrisburg
2.	Electric Reliability Council	Mid-Atlantic Area Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2535
5.	Design Electrical Rating (MWE Net)	819
6.	Date of Initial Criticality	June 5, 1974
7.	Date of Commercial Operation	September 2, 1974
8.	Condenser Cooling Method	Closed
9.	Condenser Cooling Water Source	Susquehanna River

Utility & Contractor Information

10.	Licensee	Metropolitan Edison Company
11.	Corporate Address	P. O. Box 542 Reading, Pennsylvania
12.	Corporate Contact	R. C. Arnold, Sr. Vice-President 260 Cherry Hill Rd. Parsippany, NJ 07054
13.	Architect/Engineer	Gilbert Associates, Inc.
14.	Nuclear Steam Supply System	B&W
15.	Constructor	UE&C

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Section Chief	A. N. Fasano
19.	NRC-TMI Support Staff	D. Haverkamp (717) 782-3950 FTS: 590-3950
20.	Licensing Project Manager	D. C. Dilanni / H. Silver (X27435)
21.	Docket Number	50-289
22.	License Number/Date of Issue	DPR-50, 4/19/74
23.	Public Document Room	Government Publications Section State Library of Pennsylvania Education Building Commonwealth and Walnut Street Harrisburg, Pennsylvania 17126

INSPECTION STATUS

ENFORCEMENT STATUS

```
I N S P E C T I O N   S T A T U S

OTHER ITEMS OF INTEREST

+   SYSTEMS AND COMPONENT PROBLEMS:

+   COMPLETE REQUIRED MODIFICATIONS PkiOR TO REsTART.

    PLANT STATUS:

    The plant remains shutdown, by NRC Order, pending
    completion of modifications and other actions
    related to the TMI-2 accident.

+ LAST IE SITE INSPECTION DATE:  9/23-25/80

+        INSPECTION REPORT NO:  50-289/80-28

    SUMMARY

    INSPECTION SUMMARY

+   50-289/80-07 - MAR 21 AND JUL 31:  INVESTIGATION BY THREE REGION-
    BASED INSPECTORS OF:  ANNOUNCED MEETING WITH GPU REPRESENTATIVES
    TO FOLLOW UP PREVIOUS ANNOUNCED MEETING DATED MARCH 21, 1980, TO
    INVESTIGATE AN ALLEGATION CONCERNING POSSIBLE MIS-MARKING OF ASTM:
    A312 GRADE 304 WELDED STAINLESS STEEL PIPE.  NO ITEMS OF NONCOM-
    PLIANCE WERE IDENTIFIED.

+   50-289/80-08  - MAR 17 - APR 30:  SPECIAL INSPECTION BY PROGRAM
    OFFICE STAFF OF:  HEALTH PHYSICS AND ENVIRONMENTAL AREAS; SPENT
    RESIN TANK OVERFLOW EVENT AND LICENSEE ACTION ON 'E BULLETIN NO.
    79-19.  NO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED.

+   50-289/80-16  - JUN 12, 17, 30 AND JU' 1-3, 7-9:  ROUTINE, AN-
    NOUNCED INSPECTION BY TWO REGION-BASED INSPECTORS AND THE SENIOR
    RESIDENT INSPECTOR (166 HRS) OF THE QUALITY ASSURANCE PROGRAM AND
    ITS IMPLEMENTATION IN THE AREAS OF DESIGN CHANGES/MODIFICATIONS:
    DRAWING/DOCUMENT CONTROL; RECORDS; AND, PREVIOUSLY IDENTIFIED
    ITEMS.  NO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED.

+   50-289/80-19  -  JUL 23-25:  SPECIAL, ANNOUNCED INSPECTION (132
    HRS) BY AN NRC:NRR/IE TEAM (6 PERSONS) OF UTILITY MANAGEMENT AND
    TECHNICAL COMPETENCE IN THE AREAS OF:  SHIFT TECHNICAL ADVISORS;
    STAFFING FOR STARTUP TESTING PROGRAM; ON-SITE TECHNICAL SUPPORT
    CENTER; ON-SITE OPERATIONAL SUPPORT CENTER; INDEPENDENT SAFETY
    REVIEWS; OFF-SITE AND ON-SITE STAFFING; DISSEMINATION OF OPERA-
    TING EXPERIENCES; AND, COMMUNICATIONS WITH NRC.  NO ITEMS OF
    NONCOMPLIANCE WERE IDENTIFIED.

+   50-289/80-20  - JUL 11 - AUG 15:  ROUTINE, UNANNOUNCED INSPECTION
    BY FOUR REGION BASED INSPECTORS (115 HRS) OF WORK ACTIVITIES AND
    RECORDS ASSOCIATED WITH THE RESTART PROGRAM AS WELL AS CERTAIN
    PREVIOUSLY UNRESOLVED ITEMS INCLUDING:  THE TENDON SURVEILLANCE
    PROGRAM, THE BUTT-SPLICE PROGRAM (TASK RM-1), VERIFICATION OF
    COMPONENTS PER IE BULLETIN 79-01B, IMPLEMENTATION OF IE BULLETIN
    80-11, REVIEW OF UNRESOLVED ITEMS ASSOCIATED WITH THE LICENSEE'S
    ARCHITECT-ENGINEER EFFORT, AND REVIEW OF UNRESOLVED ITEMS RE-
    LATED TO 10 CFR PAR" 21 AND CERTIFICATES OF CONFORMANCE.  NO ITEMS
    OF NONCOMPLIANCE WERE IDENTIFIED.

+   50-289/80-25  - AUG 25-29:  ROUTINE, ANNOUNCED INSPECTION BY ONE
    REGION-BASED INSPECTOR (40 HRS) OF LICENSEE'S ACTION IN RESPONSE
    TO IE BULLETIN 80-11, MASONRY WALLS.  THE INSPECTOR REVIEWED
    DOCUMENTS, HELD DISCUSSIONS WITH LICENSEE PERSONNEL, AND CON-
    DUCTED VISUAL INSPECTION OF MASONRY WALLS.  NO ITEMS OF NONCOM-
    PLIANCE WERE IDENTIFIED.
```

REPORTS RECEIVED FROM LICENSEE

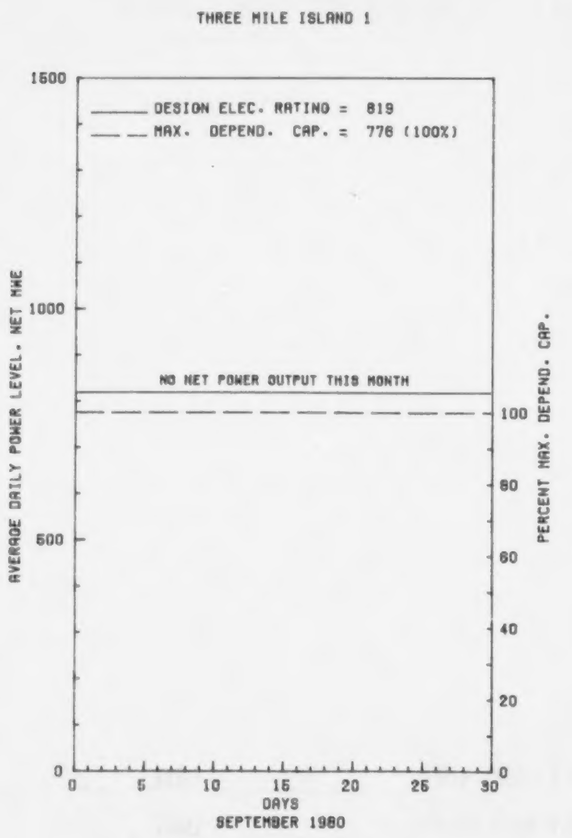
```
R E P O R T S   F R O M   L I C E N S E E
==============================================================================

  NUMBER     DATE OF     DATE OF     SUBJECT
              EVENT      REPORT
==============================================================================

  NONE THIS PERIOD.
==============================================================================
```

REVIEWED BY OMPA LOEB: P E Bohe **DATE:** 11/5/80

REVIEWED BY NRR DL: **DATE:** 11/6/80

THREE MILE ISLAND 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-289
2. Utility Contact: D. G. Mitchell (215) 921-6579
3. Licensed Thermal Power (MWt): 2535
4. Nameplate Rating (Gross MWe): 968 x 0.9 = 871
5. Design Electrical Rating (Net MWe): 819
6. Maximum Dependable Capacity (Gross MWe): 840
7. Maximum Dependable Capacity (Net MWe): 776
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	6,575.0	53,304.0
12. Hours Reactor Was Critical	.0	.0	31,731.8
13. Reactor Reserve Shutdown Hours	.0	.0	838.5
14. Hours Generator On-Line	.0	.0	31,180.9
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	0	76,531,071
17. Gross Electrical Energy (MWH)	0	0	25,484,330
18. Net Electrical Energy (MWH)	0	0	23,840,053
19. Unit Service Factor	.0	.0	58.5
20. Unit Availability Factor	.0	.0	58.5
21. Unit Capacity Factor (MDC Net)	.0	.0	57.6
22. Unit Capacity Factor (DER Net)	.0	.0	54.6
23. Unit Forced Outage Rate	100.0	100.0	32.1

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
1	02/17/79	F	720.0	D	4				REGULATORY RESTRAINT ORDER.

SUMMARY

THREE MILE ISLAND 1 REMAINS SHUTDOWN FOLLOWING THE ACCIDENT AT THREE MILE ISLAND 2.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

THREE MILE ISLAND 2

NRC POWER RESTRICTIONS: SEE "LICENSE STATUS"

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Pennsylvania/Dauphin 10 miles SE of Harrisburg
2.	Electric Reliability Council	Mid-Atlantic Area Council
3.	Type of Reactor	PWR
4.	Licenses Power Level (MWT)	2772
5.	Design Electrical Rating (MWE Net)	906
6.	Date of Initial Criticality	3/28/78
7.	Date of Commercial Operation	12/30/78
8.	Condenser Cooling Method	CLOSED
9.	Condenser Cooling Water Source	Susquehanna River

Utility & Contractor Information

10.	Licensee	Metropolitan Edison Company
11.	Corporate Address	P. O. Box 542 Reading, PA 19603
12.	Corporate Contact	R. C. Arnold, Sr. Vice-President 100 Interpace Parkway Parsippany, NJ 07054
13.	Architect/Engineer	Burns & Roe
14.	Nuclear Steam Supply System	B&W
15.	Constructor	UE&C

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Section Chief	A. N. Fasano FTS: 590-1123 (717) 948-1123
19.	IE Resident Inspector	R. Conte FTS: 590-1125 (717) 948-1125
20.	TMI Program Director	B. Snyder
21.	Docket Number	50-320
22.	License Number/Date of Issue	DPR-73/2; 2/8/78
23.	Public Document Room	Government Publications Section State Library of Pennsylvania Education Building Commonwealth & Walnut Streets Harrisburg, Pennsylvania 17126

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEM AND COMPONENT PROBLEMS:

Cleanup operations from 3/28/79 accident.

+ MANAGERIAL ITEMS:

+ NRC APPROVAL OF THE LICENSEE'S REORGANIZATION INTO THE GENERAL PUBLIC UTILITIES NUCLEAR GROUP (GPUNG) OCCURRED ON SEPTEMBER 15, 1980.

PLANT STATUS:

Decay heat is being removed by cyclic natural circulation through the "A" Once Through Steam Generator and to the reactor building ambient from both RCS loops.

+ LAST IE SITE INSPECTION DATE: 9/15/80

+ INSPECTION REPORT NO: 50-320/80-16

SUMMARY

INSPECTION SUMMARY

+ 50-320/80-05 - MAR 17 - APR 30: SPECIAL INSPECTION BY PROGRAM OFFICE STAFF OF: SELECTED NEW AND REVISED PROCEDURES SUBMITTED FOR NRC APPROVAL; HEALTH PHYSICS AND ENVIRONMENTAL AREAS; HIGH AIRBORNE ACTIVITY EVENT; AND LICENSEE ACTION ON IE BULLETIN 79-19. NO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED.

LICENSE STATUS

By Order for Modification of License dated July 20, 1979, the licensee's authority to operate the facility was suspended and the licensee's authority was limited to maintenance of the facility in the present shutdown cooling mode (44 F.R. 45271).

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-015/03L	04/28/80	05/28/80	BORIC ACID MIX TANK CONCENTRATION OUT OF SPEC DUE TO EVAPORATION FROM USE OF AIR SPARGER MIXING - MIXER BEING REPAIRED

REVIEWED BY OMPA LOEB: **DATE:** 11/5/80

REVIEWED BY NRR TMIPO: D. J. Brinkman **DATE:** 11/5/80

THREE MILE ISLAND 2

AVERAGE DAILY POWER LEVEL (MWe)

OPERATING STATUS

1. Docket Number: 50-320
2. Utility Data Prepared By: R. A. Lengel (215) 921-6581
3. Licensed Thermal Power (MWt): 2772
4. Nameplate Rating (Gross MWe): 1068 X 0.9 = 961
5. Design Electrical Rating (Net MWe): 906
6. Maximum Dependable Capacity (Gross MWe): 0
7. Maximum Dependable Capacity (Net MWe): 0
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons: None
9. Power Level To Which Restricted, If Any (Net MWe): None
10. Reasons For Restrictions, If Any:

	This Month	Yr-to
11. Hours In Reporting Period		
12. Number Of Hours Reactor Was Critical		
13. Reactor Reserve Shutdown Hours		
14. Hours Generator On-Line		
15. Unit Reserve Sh		
16. Gro		
(MWH)		
19		
20. U Availability Factor		
21. Unit Capacity Factor (Using MDC Net) 1/		
22. Unit Capacity Factor (Using DER Net) 1/		
23. Unit Forced Outage Rate		

NO DECISION ON FUTURE OPERATION
SHUTDOWN INDEFINITELY

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): None

25. If Shut Down At End Of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence

SEE SUMMARY BELOW.

SUMMARY

THE UNIT HAS REMAINED SHUTDOWN SINCE THE 03/28/79 ACCIDENT, WHICH CAUSED EXTENSIVE CORE DAMAGE.

AUTHORITY TO OPERATE FACILITY SUSPENDED BY NRC ORDER DATED JULY 20, 1979, WHICH REQUIRES LICENSEE TO MAINTAIN FACILITY IN A SHUTDOWN CONDITION.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

TROJAN

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Oregon/Columbia 42 miles N of Portland
2.	Electric Reliability Council	Western Systems Coordinating Council
3.	Type of Reactor	PWR
4.	Licenses Power Level (MWT)	3411
5.	Design Electrical Rating (MWE Net)	1130
6.	Date of Initial Criticality	December 15, 1975
7.	Date of Commercial Operation	May 20, 1976
8.	Condenser Cooling Method	Natural draft cooling tower
9.	Condenser Cooling Water Source	Columbia River

Utility & Contractor Information

10.	Licensee	Portland General Electric Co.
11.	Corporate Address	121 S. W. Salmon St. Portland, Oregon 97204
12.	Corporate Contact	C. Goodwin, Jr., Asst. V.P.
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Multiple

Regulatory Information

16.	IE Region responsible	V
17.	IE Branch Chief	J. L. Crews
18.	IE Section Chief	D. M. Sternberg
19.	IE Resident Inspector	M. H. Malmros (503) 556-3751
20.	Licensing Project Manager	C. M. Trammell
21.	Docket Number	50-344
22.	License Number/Date of Issue	NPF-1, 11/21/75
23.	Public Document Room	Columbia County Courthouse Law Library Circuit Court Room St. Helens, Oregon 97501

INSPECTION STATUS

ENFORCEMENT STATUS

FAILURE OF SOME REQUIRED CCTV'S AT PROTECTED AREA PERIMETER TO BE OPERATIONAL.
(8015 INFRACTION)

FACILITY OPERATING LICENSE NO. NPF-1 STATES IN SECTION 2.C(7) THAT, EXCEPT FOR THE INSTALLATION OF PIPE SUPPORTS, "...ALL ACTIVITIES TO WHICH A QUALITY ASSURANCE PROGRAM IS APPLICABLE SHALL, AFTER THE DATE OF ISSUE OF THIS LICENSE, BE CONDUCTED IN ACCORDANCE WITH THE QUALITY ASSURANCE PROGRAM FOR OPERATION." QUALITY ASSURANCE PROCEDURE QAP-5, "PROCEDURES, INSTRUCTIONS AND DRAWINGS," STATES IN SECTION II.A THAT "ACTIVITIES AFFECTING QUALITY SHALL BE ACCOMPLISHED IN ACCORDANCE WITH CURRENT PLANT PROCEDURES OR INSTRUCTIONS INCLUDING BUT NOT LIMITED TO APPROVED PLANT OPERATING MANUAL PROCEDURES, TEMPORARY PROCEDURES, MAINTENANCE REQUEST SPECIAL INSTRUCTIONS AND LOWER TIER PROCEDURES." THE PROCEDURE FOR STEAM GENERATOR CHANNEL HEAD DECONTAMINATION (PLANT PROCEDURE MP-T-7) REQUIRES IN STEP NO. 7.4 THAT REACTOR COOLANT SYSTEM (RCS) BORON SAMPLES BE TAKEN AT LEAST EVERY FOUR HOURS OF SPRAY FLOW OPERATION. CONTRARY TO THE REQUIREMENTS, CHEMISTRY LOGS EXAMINED BY THE INSPECTOR SHOWED THAT NO RCS BORON SAMPLES WERE TAKEN BETWEEN 0030 ON MAY 28 AND 0035 ON MAY 29, 1980, DURING WHICH PERIOD SEVERAL HOURS OF SPRAY FLOW DECONTAMINATION TOOK PLACE, WITH THE RESULT THAT RCS BORON CONCENTRATION WAS FOUND TO BE LESS THAN THE MINIMUM OF 2000 PPM REQUIRED BY TECHNICAL SPECIFICATIONS.
(8020 INFRACTION)

INSPECTION STATUS

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

PURSUANT TO THE ASLB DECISION OF JULY 11, 1980, A LICENSE AMENDMENT WAS ISSUED ON JULY 25, 1980 AUTHORIZING THE LICENSEE TO MODIFY THE CONTROL BUILDING TO RESTORE THE INITIAL DESIGN MARGINS.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS

+ THE PLANT BEGAN AND ENDED THE MONTH AT FULL POWER. NO SHUTDOWNS OCCURRED THIS MONTH.

LAST IE SITE INSPECTION DATE: 09/02-30/80+

INSPECTION REPORT NO: 50-344/80-24+

SUMMARY

INSPECTION SUMMARY:

+ INSPECTION ON JULY 7-18, 1980 (REPORT NO. 50-344/80-16) SPECIAL INSPECTION; REPORT BEING PREPARED.

+ INSPECTION ON AUGUST 1-31, 1980 (REPORT NO. 50-344/80-21) AREAS INSPECTED: ROUTINE INSPECTIONS OF PLANT OPERATIONS; PLANT MODIFICATIONS; SURVEILLANCE TESTING; PHYSICAL SECURITY; MAINTENANCE AND FOLLOWUP ON LICENSEE EVENT REPORTS. THE INSPECTION INVOLVED 163 INSPECTOR HOURS ONSITE BY THE NRC RESIDENT INSPECTORS.

RESULTS: NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

+ INSPECTION ON SEPTEMBER 8-12, 1980 (REPORT NO. 50-344/80-22) AREAS INSPECTED: ROUTINE, UNANNOUNCED INSPECTION BY REGIONAL BASED INSPECTOR OF FACILITY MODIFICATION ACTIVITIES AND ACTIVITIES RELATED TO IE BULLETINS 79-13 AND 80-08. THE INSPECTION INVOLVED 23 INSPECTOR HOURS ONSITE AND 7 INSPECTOR HOURS IN THE CORPORATE OFFICES BY ONE NRC INSPECTOR.

RESULTS: NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

+ INSPECTION ON OCTOBER 20-24, 1980 (REPORT NO. 50-344/80-23) REPORT TO BE PREPARED.

+ INSPECTION ON SEPTEMBER 2-30, 1980 (REPORT NO. 50-344/80-24) AREAS INSPECTED: ROUTINE INSPECTIONS OF PLANT OPERATIONS, SURVEILLANCE TESTING, PHYSICAL SECURITY, MAINTENANCE, CORRECTIVE ACTIONS, AND FOLLOWUP ON LICENSEE EVENT REPORTS AND ITEMS OF NONCOMPLIANCE. THE INSPECTION INVOLVED 190 INSPECTOR HOURS ONSITE BY THE NRC RESIDENT INSPECTORS.

RESULTS: NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-28	07-21-80	08-28-80	SEISMIC INSTRUMENT INOPERABLE MORE THAN 30 DAYS (SPECIAL REPORT).
80-04	09-10-80	10-04-80	PLANNED INOPERABILITY OF CABLE SPREADING ROOM SPRINKLER SYSTEM (SPECIAL REPORT).
80-05/ 01T-1	04-11-80	08-29-80	FOLLOWUP REPORT ON MSIV CLOSURE PROBLEM.
80-17/ 03L-0	08-03-80	08-29-80	INOPERABLE FAN IN WEST CONTAINMENT SPRAY PUMP ROOM.

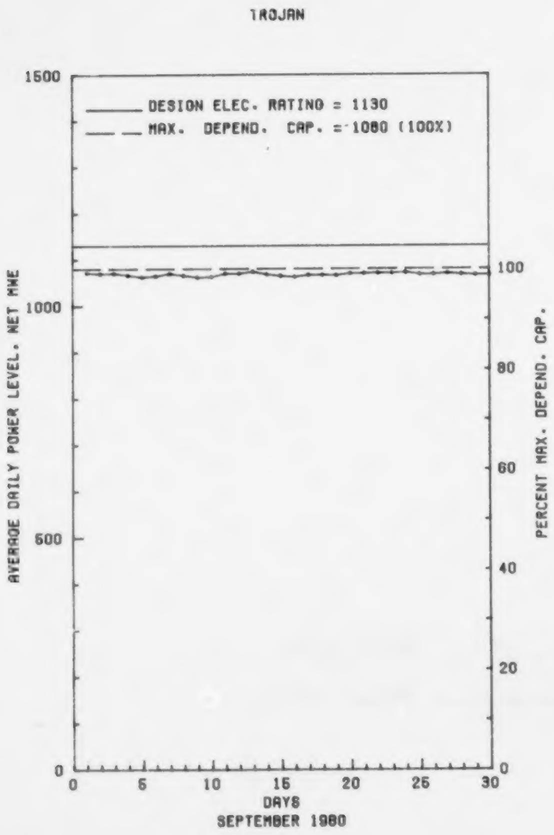
==: ===

REVIEWED BY OMPA LOEB: John L Crooks **DATE:** 11/5/80

REVIEWED BY NRR DL: Colburn **DATE:** 11/6/80

TROJAN

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-344
2. Utility Contact: G. G. Bair (503) 556-3713 x234
3. Licensed Thermal Power (MWt): 3411
4. Nameplate Rating (Gross MWe): 1280 x 0.95 = 1216
5. Design Electrical Rating (Net MWe): 1130
6. Maximum Dependable Capacity (Gross MWe): 1122
7. Maximum Dependable Capacity (Net MWe): 1080
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	6,575.0	35,783.0
12. Hours Reactor Was Critical	720.0	4,257.4	20,555.4
13. Reactor Reserve Shutdown Hours	.0	.0	2,171.8
14. Hours Generator On-Line	720.0	4,163.3	19,791.1
15. Unit Reserve Shutdown Hours	.0	.0	1,508.7
16. Gross Thermal Energy (MWH)	2,456,611	12,436,738	61,332,348
17. Gross Electrical Energy (MWH)	804,510	4,018,825	20,004,240
18. Net Electrical Energy (MWH)	768,999	3,812,036	18,845,353
19. Unit Service Factor	100.0	63.3	55.3
20. Unit Availability Factor	100.0	63.3	59.5
21. Unit Capacity Factor (MDC Net)	98.9	53.7	48.8
22. Unit Capacity Factor (DER Net)	94.5	51.3	46.6
23. Unit Forced Outage Rate	.0	13.9	27.6

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	NONE								

SUMMARY

TROJAN OPERATED AT NEAR FULL POWER FOR THE ENTIRE MONTH WITH NO OUTAGES OR REDUCTIONS.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

TURKEY POINT 3

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1. Location (state/county) (nearest population center)	Florida/Dade 25 miles S of Miami
2. Electric Reliability Council	Southeastern Electric Reliability Council
3. Type of Reactor	PWR
4. Power Level (MWT)	2200
5. Design Electrical Rating (MWE Net)	693
6. Date of Initial Criticality	October 20, 1972
7. Date of Commercial Operation	December 14, 1972
8. Condenser Cooling Method	Recirculating Canal System
9. Condenser Cooling Water Source	Canal System

Utility & Contractor Information

10. Licensee	Florida Power and Light Company
11. Corporate Address	9250 West Flagler Street P. O. Box 013100 Miami, Florida 33101
12. Corporate Contact	R. E. Uhrig, V.P. Nuclear Affairs
13. Architect/Engineer	Bechtel
14. Nuclear Steam Supply System	Westinghouse
15. Constructor	Bechtel

Regulatory Information

16. IE Region responsible	II
17. IE Branch Chief	R. C. Lewis
18. IE Section Chief	R. D. Martin
19. IE Resident Inspector	R. J. Vogt-Lowell (305) 245-7669
20. Licensing Project Manager	M. Grotenhuis (301) 492-7128
21. Docket Number	50-250
22. License Number/Date of Issue	DPR-31, 7/19/72
23. Public Document Room	Environmental and Urban Affairs Library Florida International University Miami, Florida 33199

INSPECTION STATUS

ENFORCEMENT STATUS

AS REQUIRED BY 10 CFR 20.105.B(1), "NO LICENSE SHALL POSSESS, USE OR TRANSFER LICENSED MATERIAL IN SUCH A MANNER AS TO CREATE IN ANY UNRESTRICTED AREA ... RADIATION LEVELS WHICH, IF AN INDIVIDUAL WERE CONTINUOUSLY PRESENT IN THE AREA, COULD RESULT IN HIS RECEIVING A DOSE IN EXCESS OF TWO MILLIREMS IN ANY ONE HOUR...". CONTRARY TO THE ABOVE, ON MAY 8, 1980, THE RADIATION LEVELS ON THE RADWASTE DISCHARGE PIPING IN THE UNRESTRICTED AREA WAS AT 3.5 MILLIREM/HR FOR AT LEAST ONE HOUR.
(8017 INFRACTION)

AS REQUIRED BY 10 CFR 20.203(F)(1) EACH CONTAINER OF LICENSED MATERIAL SHALL BEAR A DURABLE, CLEARLY VISIBLE LABEL IDENTIFYING THE RADIOACTIVE CONTENTS. CONTRARY TO THE ABOVE, ON MAY 8, 1980, UNMARKED DRAWER FOUND IN THE CHEMISTRY LAB HAD NUMEROUS REACTOR COOLANT SLUDGE SAMPLES WHICH CONTAINED GREATER THAN APPENDIX C QUANTITIES OF RADIOACTIVE MATERIAL. ONE SAMPLE HAD A CONTACT READING OF APPROXIMATELY 200 MILLIREM/HOUR.
(8017 DEFICIENCY)

AS REQUIRED BY 10 CFR 20.201.(B) "EACH LICENSEE SHALL MAKE OR CAUSE TO BE MADE SUCH SURVEYS AS MAY BE NECESSARY FOR HIM TO COMPLY WITH THE REGULATIONS IN THIS PART." CONTRARY TO THE ABOVE, ON MAY 8, 1980, SURVEYS WERE NOT PERFORMED TO ASSURE COMPLIANCE WITH 10 CFR 20.207(B) IN THAT LICENSEABLE QUANTITIES OF RADIOACTIVE MATERIAL WERE FOUND UNATTENDED AND UNCONTROLLED IN THE SOUTH EAST SECTOR OF THE BACK YARD BEYOND THE PROTECTED AREA FENCE, AND, FURTHER RADIATION LEVELS EXCEEDING THE REQUIREMENTS OF 10 CFR 20.105(B)(1) WERE FOUND IN THE ELECTRICAL VAULTS IN THE UNRESTRICTED AREA.
(8017 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

NORMAL OPERATION AT 100% POWER.

+ LAST IE SITE INSPECTION DATE: SEPTEMBER 23-26, 1980

+ INSPECTION REPORT NO: 50-250/80-29

SUMMARY

INSPECTION SUMMARY

+ NO REPORTS RELEASED THIS MONTH.

REPORTS RECEIVED FROM LICENSEE

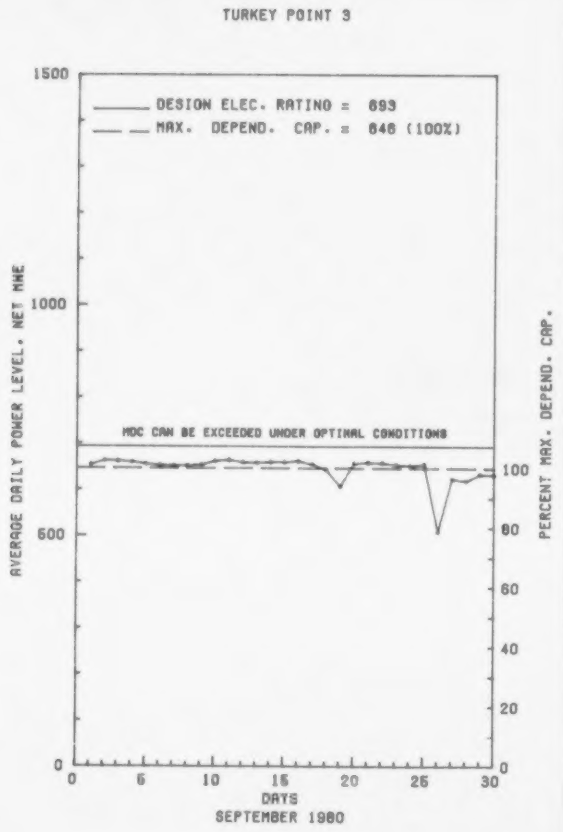
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-015/03L-0	08/02/80	09/02/80	SAFETY INJECTION PUMP FAILED TO START 4160V CIRCUIT BREAKER INOPERATIVE.
80-016/03L-0	08/13/80	09/12/80	RADIATION MONITORS R-11 AND R-12 WERE INOPERABLE DUE TO A BLOWER FAILURE.

REVIEWED BY OMPA LOEB: Brooks **DATE:** 11/5/80

REVIEWED BY NRR DL: Grotenhuis **DATE:** 11/6/80

TURKEY POINT 3

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-250
2. Utility Contact: V. T. Chilson (305) 552-3824
3. Licensed Thermal Power (MWt): 2200
4. Nameplate Rating (Gross MWe): 894 x 0.85 = 760
5. Design Electrical Rating (Net MWe): 693
6. Maximum Dependable Capacity (Gross MWe): 680
7. Maximum Dependable Capacity (Net MWe): 646
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

ITEM 21 CALCULATED BY USING A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	6,575.0	68,576.6
12. Hours Reactor Was Critical	719.2	5,696.2	52,965.6
13. Reactor Reserve Shutdown Hours	.0	.0	213.3
14. Hours Generator On-Line	717.8	5,527.4	51,217.6
15. Unit Reserve Shutdown Hours	.0	.0	121.8
16. Gross Thermal Energy (MWH)	1,558,594	11,968,962	103,437,596
17. Gross Electrical Energy (MWH)	489,285	3,764,700	32,845,700
18. Net Electrical Energy (MWH)	464,674	3,568,952	31,091,046
19. Unit Service Facto.	99.7	84.1	74.7
20. Unit Availability Factor	99.7	84.1	74.9
21. Unit Capacity Factor (MDC Net)	99.9	84.0	70.5
22. Unit Capacity Factor (DER Net)	93.1	78.3	65.4
23. Unit Forced Outage Rate	.3	1.2	2.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): STEAM GEN. TUBE INSPECTION PROGRAM - OCT. 6 - NOV. 1, 1980.*
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

*REFUELING, MAINTENANCE AND INSPECTIONS - MARCH 1 - MAY 1? 1981.

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
17	09/18/80	S	.0	B	5		HA	VALVEX	LOAD REDUCTION TO PERFORM PERIODIC TEST OF TURBINE MAIN STEAM STOP, REHEAT STOP, AND REHEAT INTERCEPT VALVES.
18	09/26/80	F	2.2	A	3		IA	ELECON	UNIT NO. 3 WAS TRIPPED BY REACTOR PROTECTION SYSTEM DUE TO SPURIOUS SIGNAL FROM NUCLEAR INSTRUMENTATION SYSTEM CHANNEL N-41 WHILE CHANNEL N-43 WAS IN TRIPPED MODE.

SUMMARY

TURKEY POINT 3 OPERATED AT FULL POWER FOR THE ENTIRE MONTH, WITH ONE REDUCTION FOR MAINTENANCE ON THE 18TH, AND ONE BRIEF OUTAGE ON THE 26TH DUE TO A SPURIOUS RPS SIGNAL.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

TURKEY POINT 4

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Florida/Dade 25 miles S of Miami
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2200
5.	Design Electrical Rating (MWE Net)	693
6.	Date of Initial Criticality	June 11, 1973
7.	Date of Commercial Operation	September 7, 1973
8.	Condenser Cooling Method	Recirculating Canal System
9.	Condenser Cooling Water Source	Canal System

Utility & Contractor Information

10.	Licensee	Florida Power and Light Company
11.	Corporate Address	9250 West Flagler Street P. O. Box 013100 Miami, Florida 33101
12.	Corporate Contact	R. E. Uhrig, V. P. Nuclear Affairs
13.	Architect/Engineer	Bechtal
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. Lewis
18.	IE Section Chief	R. D. Martin
19.	IE Resident Inspector	A. J. IGNATONIS (305) 245-7669
20.	Licensing Project Manager	M. Grotenhuis (301) 492-7128
21.	Docket Number	50-251
22.	License Number/Date of Issue	DPR-41, 4/10/73
23.	Public Document Room	Environmental and Urban Affairs Library Florida International University Miami, Florida 33199

INSPECTION STATUS

ENFORCEMENT STATUS

AS REQUIRED BY 10 CFR 20.105.B(1), "NO LICENSE SHALL POSSESS, USE OR TRANSFER LICENSED MATERIAL IN SUCH A MANNER AS TO CREATE IN ANY UNRESTRICTED AREA ... RADIATION LEVELS WHICH, IF AN INDIVIDUAL WERE CONTINUOUSLY PRESENT IN THE AREA, COULD RESULT IN HIS RECEIVING A DOSE IN EXCESS OF TWO MILLIREMS IN ANY ONE HOUR...". CONTRARY TO THE ABOVE, ON MAY 8, 1980, THE RADIATION LEVELS ON THE RADWASTE DISCHARGE PIPING IN THE UNRESTRICTED AREA WAS AT 3.5 MILLIREM/HR FOR AT LEAST ONE HOUR.
(8015 INFRACTION)

AS REQUIRED BY 10 CFR 20.203(F)(1) EACH CONTAINER OF LICENSED MATERIAL SHALL BEAR A DURABLE, CLEARLY VISIBLE LABEL IDENTIFYING THE RADIOACTIVE CONTENTS. CONTRARY TO THE ABOVE, ON MAY 8, 1980, UNMARKED DRAWER FOUND IN THE CHEMISTRY LAB HAD NUMEROUS REACTOR COOLANT SLUDGE SAMPLES WHICH CONTAINED GREATER THAN APPENDIX C QUANTITIES OF RADIOACTIVE MATERIAL. ONE SAMPLE HAD A CONTACT READING OF APPROXIMATELY 200 MILLIREM/HOUR.
(8015 DEFICIENCY)

AS REQUIRED BY 10 CFR 20.201.(B) "EACH LICENSEE SHALL MAKE OR CAUSE TO BE MADE SUCH SURVEYS AS MAY BE NECESSARY FOR HIM TO COMPLY WITH THE REGULATIONS IN THIS PART." CONTRARY TO THE ABOVE, ON MAY 8, 1980, SURVEYS WERE NOT PERFORMED TO ASSURE COMPLIANCE WITH 10 CFR 20.207(B) IN THAT LICENSEABLE QUANTITIES OF RADIOACTIVE MATERIAL WERE FOUND UNATTENDED AND UNCONTROLLED IN THE SOUTH EAST SECTOR OF THE BACK YARD BEYOND THE PROTECTED AREA FENCE, AND, FURTHER RADIATION LEVELS EXCEEDING THE REQUIREMENTS OF 10 CFR 20.105(B)(1) WERE FOUND IN THE ELECTRICAL VAULTS IN THE UNRESTRICTED AREA.
(8015 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

NORMAL OPERATIONS.

+ LAST IE SITE INSPECTION DATE: SEPTEMBER 23-26, 1980

+ INSPECTION REPORT NO: 50-251/80-28

SUMMARY

INSPECTION SUMMARY

+ NO REPORTS RELEASED THIS MONTH.

REPORTS RECEIVED FROM LICENSEE

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-011/03L-0	08/01/80	09/02/80	FIRE DETECTOR IN A ROOM WERE FOUND INOPERATIVE.

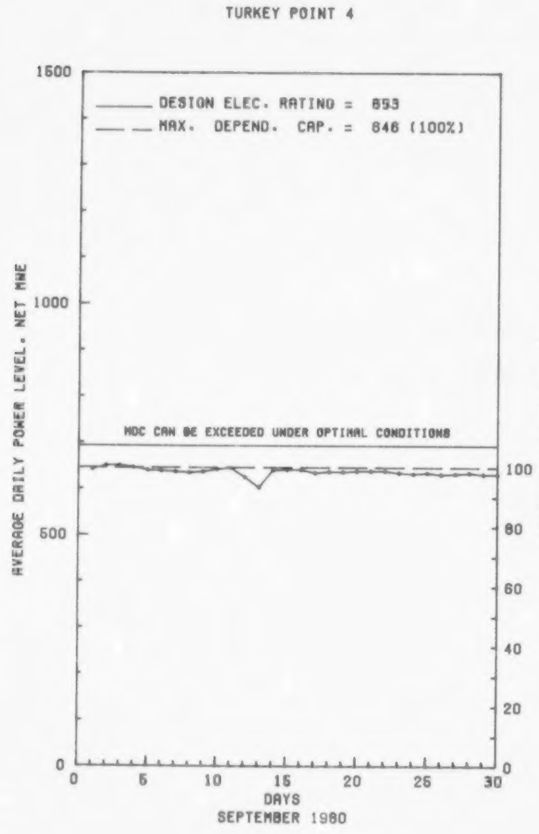
==

REVIEWED BY OMPA LOEB: John L. Crooks **DATE:** 11/5/80

REVIEWED BY NRR DL: Grotenhuis **DATE:** 11/6/80

TURKEY POINT 4

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-251
2. Utility Contact: V. T. Chilson (305) 552-3824
3. Licensed Thermal Power (MWt): 2200
4. Nameplate Rating (Gross MWe): 894 x 0.85 = 760
5. Design Electrical Rating (Net MWe): 693
6. Maximum Dependable Capacity (Gross MWe): 680
7. Maximum Dependable Capacity (Net MWe): 666
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

ITEM 21 CALCULATED BY USING A WEIGHTED AVERAGE.

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	6,575.0	67,304.0
12. Hours Reactor Was Critical	720.0	5,252.1	46,079.9
13. Reactor Reserve Shutdown Hours	0	12.3	166.6
14. Hours Generator On-Line	720.0	5,164.8	44,347.3
15. Unit Reserve Shutdown Hours	.0	12.3	31.2
16. Gross Thermal Energy (MWH)	1,578,198	11,207,334	92,426,314
17. Gross Electrical Energy (MWH)	482,585	3,479,940	29,383,418
18. Net Electrical Energy (MWH)	458,209	3,298,448	[illegible]
19. Unit Service Factor	100.0	78.6	71.2
20. Unit Availability Factor	100.0	78.7	71.2
21. Unit Capacity Factor (MDC Net)	98.5	77.7	89.4
22. Unit Capacity Factor (DER Net)	91.8	72.4	64.5
23. Unit Forced Outage Rate	.0	.3	3.0

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUELING, MAINTENANCE, AND INSPECTIONS 11/09/80 - 01/10/81
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

II

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
17	09/12/80	S	.0	B	5		HA	VALVEX	LOAD REDUCTION TO PERFORM PERIODIC TEST OF TURBINE MAIN STEAM STOP, REHEAT STOP, AND REHEAT INTERCEPT VALVES.

SUMMARY

TURKEY POINT 4 OPERATED AT OR NEAR FULL POWER FOR THE ENTIRE MONTH, WITH ONE REDUCTION FOR TESTING ON THE 12TH.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source

VERMONT YANKEE

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Vermont/Windham 5 miles S of Brattleboro
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	1593
5.	Design Electrical Rating (MWE Net)	514
6.	Date of Initial Criticality	March 24, 1972
7.	Date of Commercial Operation	November 30, 1972
8.	Condenser Cooling Method	Closed Cycle Cooling Towers
9.	Condenser Cooling Water Source	Connecticut River

Utility & Contractor Information

10.	Licensee	Vermont Yankee Nuclear Power Corp.
11.	Corporate Address	20 Turnpike Road Westboro, Massachusetts 01581
12.	Corporate Contact	D. W. Edwards Assistant to the Vice President
13.	Architect/Engineer	Ebasco
14.	Nuclear Steam Supply System	GE
15.	Constructor	Ebasco

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	IE Section Chief	T. T. MARTIN
19.	IE Resident Inspector	W. J. RAYMOND (802) 257-4310
20.	Licensing Project Manager	V. L. Rooney
21.	Docket Number	50-271
22.	License Number/Date of Issue	DPR-28, 3/21/72
23.	Public Document Room	Brooks Memorial Library 224 Main Street Brattleboro, Vermont 05301

INSPECTION STATUS

ENFORCEMENT STATUS

ISOLATION ZONE NOT MAINTAINED AS REQUIRED.
(8011 INFRACTION)

FAILURE TO IMPLEMENT KEY CONTROL PROCEDURE.
(8011 INFRACTION)

DOOR ALARM NOT TESTED AS REQUIRED.
(8011 INFRACTION)

COMMUNICATIONS DEVICE NOT TESTED AT REQUIRED FREQUENCY.
(8011 INFRACTION)

SECURITY ASSISTANCE ARRANGEMENT NOT DOCUMENTED.
(8011 DEFICIENCY)

SECURITY PROCEDURE NOT CONSISTENT WITH SECURITY PLAN.
(8011 DEFICIENCY)

MEMBER OF SECURITY FORCE NOT QUALIFIED IN ONE AREA.
(8011 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS:

◆ THE PLANT SHUTDOWN ON SEPTEMBER 25, 1980 TO BEGIN THE 1980 REFUELING OUTAGE.

◆ LAST IE SITE INSPECTION DATE: 9/29/80

◆ INSPECTION REPORT NO: 50-271/80-16

SUMMARY

INSPECTION SUMMARY

◆ 50-271/80-08 - JUN 2 - JUL 4: ROUTINE, UNANNOUNCED INSPECTION BY TWO RESIDENT INSPECTORS (61 HRS) OF: PLANT OPERATIONS; SAFETY SYSTEM OPERABILITY VERIFICATION; NRC IN-OFFICE REVIEW OF LICENSEE EVENT REPORTS; LICENSEE RESPONSE TO IE BULLETIN 80-17; AND LICENSEE TRAINING STAFF QUALIFICATIONS. NO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED.

◆ 50-271/80-11 - JUL 28 - AUG 1: UNANNOUNCED, PHYSICAL SECURITY INSPECTION BY ONE REGION BASED INSPECTOR (41 HRS). INCLUDED REVIEW OF SECURITY PLAN; SECURITY PROGRAM AUDIT; SECURITY ORGANIZATION; PHYSICAL BARRIERS (PROTECTED AREA AND VITAL AREAS); LIGHTING; ACCESS CONTROLS (IDENTIFICATION, AUTHORIZATION AND BADGING; SEARCH; ESCORTING; VITAL AREAS); ALARM STATIONS; DETECTION AIDS; ASSESSMENT AIDS; COMMUNICATIONS; TESTING AND MAINTENANCE; COMPENSATORY MEASURES; POWER SUPPLY; RESPONSE, LOCKS, KEYS, AND COMBINATIONS; RECORDS AND REPORTS. LICENSEE ACTION ON PREVIOUSLY IDENTIFIED NONCOMPLIANCE ITEMS RELATED TO PHYSICAL SECURITY: FOLLOW-UP ON IE BULLETIN 79-16 REGARDING VITAL AREA ACCESS CONTROLS AND IE CIRCULAR 80-09 REGARDING PLANT COMMUNICATIONS SYSTEMS. SEVEN ITEMS OF NONCOMPLIANCE WERE IDENTIFIED: (1) MEMBER OF SECURITY FORCE NOT QUALIFIED IN ONE AREA; (2) SECURITY PROCEDURE NOT CONSISTENT WITH SECURITY PLAN; (3) ISOLATION ZONE NOT MAINTAINED AS REQUIRED; (4) COMMUNICATIONS DEVICE NOT TESTED AS REQUIRED; (5) DOOR ALARM NOT TESTED AS REQUIRED; (6) SECURITY ASSISTANCE ARRANGEMENT NOT DOCUMENTED; AND, (7) FAILURE TO IMPLEMENT A PROCEDURE.

◆ 50-271/80-13 - AUG 11 - SEP 12: ROUTINE, ONSITE, REGULAR AND BACKSHIFT INSPECTION BY THREE RESIDENT INSPECTORS (120 HRS). AREAS INSPECTED INCLUDED: ACTIONS TAKEN ON PREVIOUS INSPECTION FINDINGS; REVIEW OF PLANT OPERATIONS, INCLUDING: INSTRUMENTATION AND ALARMS, SHIFT MANNING, RADIATION PROTECTION CONTROLS, PLANT HOUSEKEEEPING, FIRE PROTECTION/PREVENTION, CONTROL OF EQUIPMENT, AND SHIFT LOGS AND OPERATING RECORDS; SYSTEM OPERATIONAL SAFETY VERIFICATION; LICENSEE STAFFING; LICENSEE REPORTING; IE BULLETIN REVIEW; WITNESS OF SURVEILLANCE TESTS; RESPONSE TO PLANT EVENTS; OBSERVATIONS OF PHYSICAL SECURITY; PLANT MAINTENANCE, MODIFICATIONS, AND REFUELING PREPARATIONS; T-QUENCHER-WELD REPAIR; AND, PLANT COMPUTER CALCULATION OF CORE MTPF. NO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

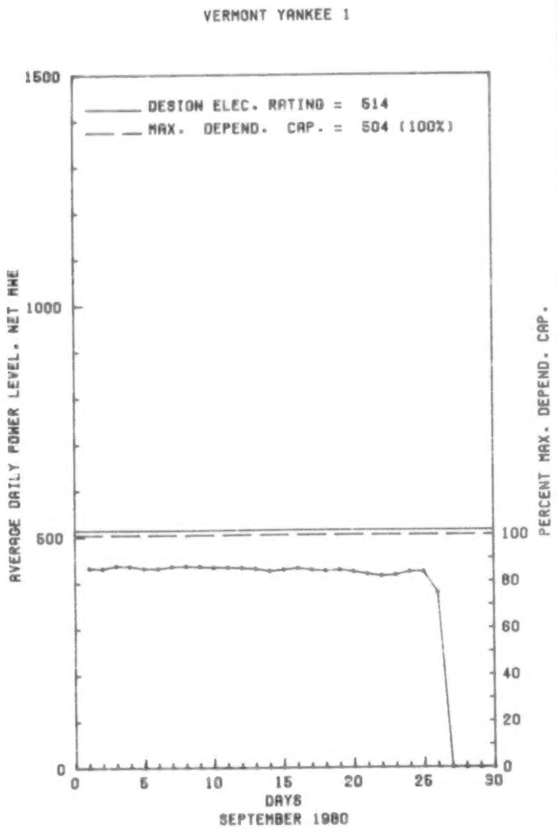
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE			

REVIEWED BY OMPA LOEB: **DATE:** 11-4-80

REVIEWED BY NRR DL: **DATE:** 11/6/80

VERMONT YANKEE

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-271
2. Utility Contact: F.C. BEERS (617) 366-9011
3. Licensed Thermal Power (MWt): 1593
4. Nameplate Rating (Gross MWe): 626 x 0.9 = 563
5. Design Electrical Rating (Net MWe): 514
6. Maximum Dependable Capacity (Gross MWe): 535
7. Maximum Dependable Capacity (Net MWe): 504
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
END OF CYCLE COASTDOWN

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	6,575.0	70,369.8
12. Hours Reactor Was Critical	624.4	6,266.6	57,425.2
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	623.5	6,200.1	55,527.9
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	869,444	9,347,970	78,572,581
17. Gross Electrical Energy (MWH)	286,043	3,136,006	25,105,676
18. Net Electrical Energy (MWH)	266,815	2,972,808	24,741,173
19. Unit Service Factor	86.6	94.3	78.9
20. Unit Availability Factor	86.6	94.3	78.9
21. Unit Capacity Factor (MDC Net)	73.5	89.7	69.8
22. Unit Capacity Factor (DER Net)	72.1	88.0	68.4
23. Unit Forced Outage Rate	.0	1.9	6.0

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 11/24/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-21	09/26/80	S	96.5	C	1		RC	FUELXX	SHUTDOWN FOR ANNUAL REFUELING.

SUMMARY

VERMONT YANKEE OPERATED AT APPROXIMATELY 85% THROUGH THE 26TH OF THE MONTH, WHEN THE UNIT SHUTDOWN FOR REFUELING.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

YANKEE ROWE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Massachusetts/Franklin 25 miles NE of Pittsfield
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	600
5.	Design Electrical Rating (MWE Net)	175
6.	Date of Initial Criticality	August 19, 1960
7.	Date of Commercial Operation	July 1961
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Deerfield River

Utility & Contractor Information

10.	Licensee	Yankee Atomic Electric Company
11.	Corporate Address	20 Turnpike Road Westboro, Massachusetts 01581
12.	Corporate Contact	D. W. Edwards, Asst. to V.P.
13.	Architect/Engineer	Stone & Webster
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Section Chief	T. T. MARTIN
19.	IE Resident Inspector	T. Foley (413) 424-5242
20.	Licensing Project Manager	R. Caruso (X27232)
21.	Docket Number	50-29
22.	License Number/Date of Issue	DPR-3, 12/24/63
23.	Public Document Room	Greenfield Community College 1 College Drive Greenfield, Massachusetts 01301

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

OTHER ITEMS OF INTEREST

PLANT STATUS:

♦ THE PLANT REMAINS IN COLD SHUTDOWN WITH A REVISED EXPECTED RESTART DATE OF LATE OCTOBER 1980.

♦ LAST IE SITE INSPECTION DATE: 9/22-25/80

♦ INSPECTION REPORT NO: 50-29/80-17

SUMMARY

INSPECTION SUMMARY

♦ 50-29/80-10 - MAY 27 - JUL 3: ROUTINE ONSITE REGULAR AND BACKSHIFT INSPECTIONS BY THE RESIDENT INSPECTOR (41 HRS) OF PLANT OPERATIONS, INCLUDING: REVIEW OF SHIFT LOGS AND OPERATING RECORDS, SURVEILLANCE TEST, PHYSICAL SECURITY, RADIATION PROTECTION CONTROLS, TOUR OF ACCESSIBLE AREAS OF THE PLANT, LICENSEE ACTION ON SELECTED PREVIOUS INSPECTION FINDINGS, REVIEW OF SELECTED BULLETINS AND ROUTINE REPORTS, AND INSTRUCTOR QUALIFICATIONS. NO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED.

♦ 50-29/80-12 - JUL 16 - AUG 6: ROUTINE, UNANNOUNCED INSPECTION ON NORMAL AND BACKSHIFTS BY THE RESIDENT INSPECTOR (57 HRS) OF PLANT OPERATIONS INCLUDING: LOG AND RECORD REVIEW; TOUR OF THE FACILITY; OBSERVATION OF PHYSICAL SECURITY; HOUSEKEEPING; RADIATION PROTECTION CONTROLS; WASTE SHIPMENT ACTIVITIES; MAINTENANCE ACTIVITIES; FOLLOWUP OF SELECTED IE BULLETINS; AND, FOLLOWUP OF SELECTED IE CIRCULARS. ONE ITEM OF NONCOMPLIANCE WAS IDENTIFIED: FAILURE TO CHECK DOORS SECURED.

♦ 50-29/80-13 - AUG 4 - AUG 6: ROUTINE, UNANNOUNCED INSPECTION BY ONE REGION-BASED INSPECTOR (22 HRS) OF THE CONTAINMENT INTEGRATED LEAK RATE TEST PROCEDURE; CONTAINMENT LEAK RATE TEST ACTIVITIES; INSERVICE TESTING PROGRAM FOR PUMPS AND VALVES; AND LICENSEE ACTION ON PREVIOUS INSPECTION FINDINGS. NO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

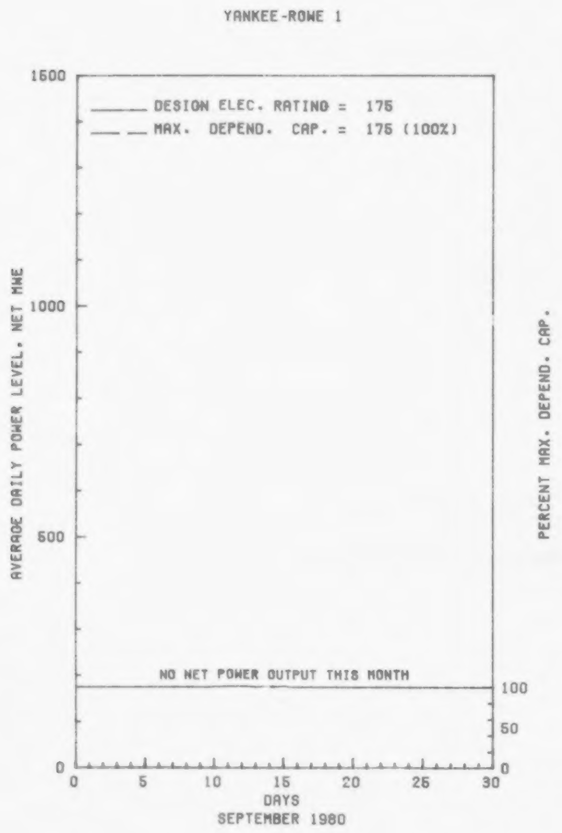
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
79-021/ 02X	09/11/79	08/18/80	CODE DEFECTS IDENTIFIED IN STEAM GENERATOR NOZZLE WELDS

REVIEWED BY OMPA LOEB: John R Crooks **DATE:** 11/5/80

REVIEWED BY NRR DL: Ralph Caruso **DATE:** 11/5/80

YANKEE ROWE

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-029
2. Utility Contact: F.C. BEERS (617) 366-9011
3. Licensed Thermal Power (MWt): 600
4. Nameplate Rating (Gross MWe): 185 x 1.0 = 185
5. Design Electrical Rating (Net MWe): 175
6. Maximum Dependable Capacity (Gross MWe): 180
7. Maximum Dependable Capacity (Net MWe): 175
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

CAPACITY FACTORS CALCULATED WITH A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	6,575.0	174,212.0
12. Hours Reactor Was Critical	.0	445.8	138,810.4
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	.0	439.7	134,467.1
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	253,769	72,002,091
17. Gross Electrical Energy (MWH)	0	80,635	22,152,788
18. Net Electrical Energy (MWH)	0	75,759	20,741,893
19. Unit Service Factor	.0	6.7	77.2
20. Unit Availability Factor	.0	6.7	77.2
21. Unit Capacity Factor (MDC Net)	.0	6.6	70.2
22. Unit Capacity Factor (DER Net)	.0	6.6	70.2
23. Unit Forced Outage Rate	100.0	91.6	4.9

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 11/01/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
1	01/19/80	F	720.0	A	4				THE PLANT REMAINS SHUTDOWN FOR TURBINE ROTOR INVESTIGATION AND REPAIR.

SUMMARY

YANKEE-ROWE CONTINUED TO REMAIN OFF-LINE FOR THE MONTH FOR ROTOR REPAIR.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

ZION 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

* 1. Location (state/countv) (nearest population center)	Illinois/Lake 40 miles N of Chicago
2. Electric Reliability Council	Mid-America Interpool Network
3. Type of Reactor	PWR
4. Power Level (MWT)	3250
5. Design Electrical Rating (MWE Net)	1040
6. Date of Initial Criticality	June 19, 1973
7. Date of Commercial Operation	December 31, 1973
8. Condenser Cooling Method	Straight Through
9. Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10. Licensee	Commonwealth Edison Co.
11. Corporate Address	P. O. Box 767 Chicago, Illinois
12. Corporate Contact	Cordell Reed, Manager of Nuclear Licensing
13. Architect/Engineer	Sargent & Lundy
14. Nuclear Steam Supply System	Westinghouse
15. Constructor	Various

Regulatory Information

16. IE Region responsible	III
17. IE Branch Chief	R. Heishman
18. IE Section Chief	R. L. Spessard
19. IE Resident Inspector	J. E. Kohler (312) 746-2313
20. Licensing Project Manager	D. Wigginton
21. Docket Number	50-295
22. License Number/Date of Issue	DPR-39 issued 4/6/73
23. Public Document Room	Zion-Benton Public Library 2600 Emmans Avenue Zion, Illinois 60099

INSPECTION STATUS

ENFORCEMENT STATUS

10 CFR 50.54 (I-1) REQUIRES THAT THE LICENSEE HAVE IN EFFECT AN OPERATOR REQUALIFICATION PROGRAM. COMMONWEALTH EDISON PROCEDURE, (ZAP 2-52, APPENDIX A, REV. 2, 1-12-79- REQUALIFICATION PROGRAM FOR LICENSED OPERATORS), REQUIRES AN OPERATOR OR SENIOR OPERATOR, RECEIVING A GRADE OF LESS THAN 80% ON ONE EXAMINATION SUBJECT OF THE REQUALIFICATION EXAMINATION, ATTEND A LECTURE ON THAT SUBJECT, FOLLOWED BY A QUIZ TO DEMONSTRATE COMPREHENSION. CONTRARY TO THE ABOVE, TWELVE OPERATORS WHO HAD RECEIVED A GRADE OF LESS THAN 80% ON ONE EXAMINATION SUBJECT OF THE REQUALIFICATION EXAMINATION DID NOT TAKE THE LECTURE QUIZ AND TEN OF THESE TWELVE DID NOT ATTEND THE SPECIFIC LECTURE ON THE SUBJECT IN WHICH THEY RECEIVED A GRADE OF BELOW 80%.
(8016 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

Unit 1 operating routinely.

LAST IE SITE INSPECTION DATE: July 30-31, 1980

INSPECTION REPORT NO: 80-18

SUMMARY

INSPECTION SUMMARY

+ No inspections conducted.

REPORTS RECEIVED FROM LICENSEE

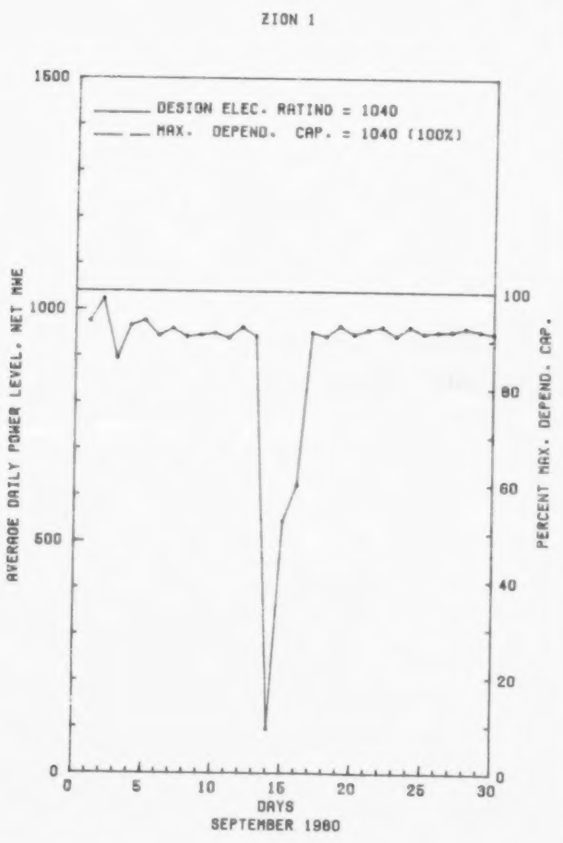
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-03/ 03L-0	08/28/80	08/29/80	2,000 gallons of wastewater were added to Lake Discharge Tank 0B while releasing it to the lake. Analysis of water released was inaccurate as a result.
80-04/ 03L-0	08/02/80	08/29/80	As a result of a valve erroneously left open, 6,000 gallons of Lake Discharge Tank 0A was inadvertently discharged to Lake.
80-06/ 01T-0	08/21/80	09/02/80	During a surveillance on Auxiliary Building ventilation system, charcoal filters were found serving Fuel Building and were not available during nonrefueling modes of operation.
80-07/ 01T-0	08/15/80	09/09/80	Hydrogen purge fan 1RV021 was made inoperable during modifications to post LOCA hydrogen recombiner piping mods.
80-37/ 03L-0	08/12/80	09/12/80	Use of an uncalibrated instrument resulted in reactor trip switches 63AST being misset 11 psig in a non-conservative direction.
80-38/ 03L-0	08/20/80	09/11/80	During normal operation the operability surveillance for the Pyra-Alarm Detectors could not be performed in prescribed time.
80-39/ 03L-0	03/03/80	09/22/80	During Mo. contain. isolation (CI) valve operability test, press. liq. space sample valve 1AOV-SS9355A failed to close.
80-40/ 03L-0	06/19/80	09/22/80	Loop D overpower/overtemp. delta T channel 1T-431 malfunctioned initiating associated alarms and bistable trips.

REVIEWED BY OMPA LOEB: John L Crooks **DATE:** 11/5/80

REVIEWED BY NRR DL: D L Wigginton **DATE:** 11/6/80

ZION 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-295
2. Utility Contact: J. COOK (312) 746-2084
3. Licensed Thermal Power (MWt): 3250
4. Nameplate Rating (Gross MWe): 1220 x 0.9 = 1098
5. Design Electrical Rating (Net MWe): 1040
6. Maximum Dependable Capacity (Gross MWe): 1085
7. Maximum Dependable Capacity (Net MWe): 1040
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	6,575.0	59,183.0
12. Hours Reactor Was Critical	701.6	5,343.1	42,450.8
13. Reactor Reserve Shutdown Hours	.0	0	2,621.8
14. Hours Generator On-Line	691.8	5,181.3	41,268.6
15. Unit Reserve Shutdown Hours	0	.0	.0
16. Gross Thermal Energy (MWH)	2,185,365	16,038,457	115,421,458
17. Gross Electrical Energy (MWH)	677,700	4,974,350	37,206,400
18. Net Electrical Energy (MWH)	649,497	4,735,954	35,220,200
19. Unit Service Factor	96.1	78.8	69.7
20. Unit Availability Factor	96.1	78.8	69.7
21. Unit Capacity Factor (MDC Net)	86.7	69.3	57.2
22. Unit Capacity Factor (DER Net)	86.7	69.3	57.2
23. Unit Forced Outage Rate	3.9	21.2	11.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUELING - JANUARY 16, 1981 - APPROX. 8 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
19	09/14/80	F	15.6	A	2				UNIT SHUTDOWN DUE TO ELECTRICAL GROUND REPAIR ON LCV-459.
20	09/15/80	F	12.6	H	3				TURBINE/REACTOR TRIP CAUSED BY CONTRACTOR ACCIDENTLY JARRING TURBINE AUTO-STOP TRIP RELAY HOUSING.

SUMMARY

ZION 1 OPERATED AT NEAR FULL POWER FOR SEPTEMBER WITH ONE OUTAGE ON THE 14TH FOR ELECTRICAL GROUND REPAIR AND ANOTHER ON THE 15TH DUE TO AN ACCIDENTAL TURBINE REACTOR TRIP.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

ZION 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Illinois/Lake 40 miles N of Chicago
2.	Electric Reliability Council	Mid-America Interpool Network
3.	Type of Reactor	PWR
4.	Licenses Power Level (MWT)	3250
5.	Design Electrical Rating (MWE Net)	1040
6.	Date of Initial Criticality	December 24, 1973
7.	Date of Commercial Operation	September 17, 1974
8.	Condenser Cooling Method	Straight Through
9.	Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10.	Licensee	Commonwealth Edison Co.
11.	Corporate Address	P. O. Box 767 Chicago, Illinois
12.	Corporate Contact	Cordell Reed, Manager of Nuclear Licensing
13.	Architect/Engineer	Sergent & Lundy
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Various

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	R. L. Spessard
19.	IE Resident Inspector	J. E. Kohler (312) 746-2313
20.	Licensing Project Manager	D. Wigginton
21.	Docket Number	50-304
22.	License Number/Date of Issue	DPR-48, 11/14/73
23.	Public Document Room	Zion-Benton Public Library 2600 Emmans Avenue Zion, Illinois 60099

INSPECTION STATUS

ENFORCEMENT STATUS

TECHNICAL SPECIFICATION 3.11.3.A REQUIRES THAT TWO SERVICE WATER PUMPS OR A CIRCULATING WATER PUMP BE OPERATIONAL ON THE DISCHARGE PATH WHEN RELEASING LIQUID RADIOACTIVE WASTES. CONTRARY TO THE ABOVE, ON JUNE 27, 1980, APPROXIMATELY 4000 GALLONS FROM THE 0A LAKE DISCHARGE TANK WERE RELEASED WITH ONLY ONE SERVICE WATER PUMP IN OPERATION. THIS EVENT WAS REPORTED BY THE LICENSEE IN LER NO. 50-304/80-23.
(8014 INFRACTION)

10 CFR 50.54 (I-1) REQUIRES THAT THE LICENSEE HAVE IN EFFECT AN OPERATOR REQUALIFICATION PROGRAM. COMMONWEALTH EDISON PROCEDURE, (ZAP 2-52, APPENDIX A, REV. 2, 1-12-79- REQUALIFICATION PROGRAM FOR LICENSED OPERATORS), REQUIRES AN OPERATOR OR SENIOR OPERATOR, RECEIVING A GRADE OF LESS THAN 80% ON ONE EXAMINATION SUBJECT OF THE REQUALIFICATION EXAMINATION, ATTEND A LECTURE ON THAT SUBJECT, FOLLOWED BY A QUIZ TO DEMONSTRATE COMPREHENSION. CONTRARY TO THE ABOVE, TWELVE OPERATORS WHO HAD RECEIVED A GRADE OF LESS THAN 80% ON ONE EXAMINATION SUBJECT OF THE REQUALIFICATION EXAMINATION DID NOT TAKE THE LECTURE QUIZ AND TEN OF THESE TWELVE DID NOT ATTEND THE SPECIFIC LECTURE ON THE SUBJECT IN WHICH THEY RECEIVED A GRADE OF BELOW 80%.
(8016 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

MANAGERIAL ITEMS

+ Effective June 26, 1980, Mr. K. Graesser, Assistant Superintendent, Quad-Cities Generating Plant became Plant Superintendent at the Zion Generating Station. (80-20)

PLANT STATUS

Unit 2 operating routinely.

+LAST IE SITE INSPECTION DATE: August 20-22, 1980

+ INSPECTION REPORT NO: 80-20

SUMMARY

INSPECTION SUMMARY

+ Inspection on August 20-22, (80-20): Facility organization and operation, measurements and inventory control, shipping and receiving, internal controls, records and reports, and management of material control system. The inspection involved 42 inspector-hours onsite by two NRC inspectors and was begun during the regular hours. The licensee was found to be in compliance with NRC requirements in the seven areas examined during the inspection.

REPORTS RECEIVED FROM LICENSEE

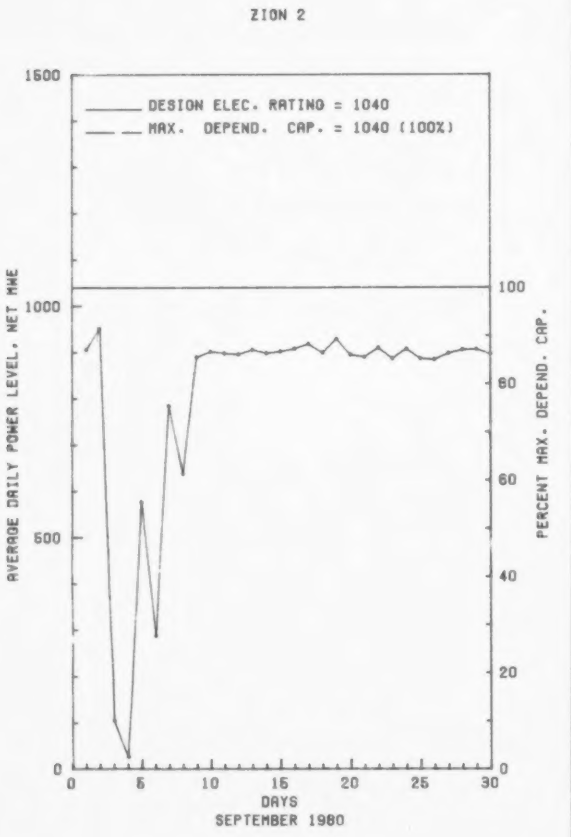
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-03	08/16/80	09/15/80	During normal operation, a low operating oil pressure resulted for the 2B MSIV as a result of oil pump failure.
80-21	07/26/80	09/11/80	During startup the 2A S/G flow channel failed to low output (non-conservative).

REVIEWED BY OMPA LOEB: John f Brooks DATE: 11/5/80

REVIEWED BY NRR DL: D L Wigginton DATE: 11/6/80

ZION 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-304
2. Utility Contact: JOAN COOK (312) 746-2084
3. Licensed Thermal Power (MWt): 3250
4. Nameplate Rating (Gross MWe): 1220 x 0.9 = 1098
5. Design Electrical Rating (Net MWe): 1040
6. Maximum Dependable Capacity (Gross MWe): 1085
7. Maximum Dependable Capacity (Net MWe): 1040
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	6,575.0	52,396.0
12. Hours Reactor Was Critical	692.5	4,061.4	37,665.0
13. Reactor Reserve Shutdown Hours	.0	.0	226.1
14. Hours Generator On-Line	670.3	3,869.5	36,683.4
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,958,742	11,524,986	104,160,302
17. Gross Electrical Energy (MWH)	608,600	3,715,880	33,437,100
18. Net Electrical Energy (MWH)	578,241	3,515,530	31,716,988
19. Unit Service Factor	93.1	58.9	69.4
20. Unit Availability Factor	93.1	58.9	69.4
21. Unit Capacity Factor (MDC Net)	77.2	51.4	57.7
22. Unit Capacity Factor (DER Net)	77.2	51.4	57.7
23. Unit Forced Outage Rate	6.9	19.2	16.1

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
22	09/03/80	F	38.8	A	3				REACTOR TRIP FROM LO-LO STEAM GENERATOR LEVEL FROM LOSS OF 2B FEEDWATER PUMP.
23	09/05/80	F	10.9	A	3				REACTOR TRIP FROM HIGH LEVEL ON "D" STEAM GENERATOR FROM 2C FEEDWATER PUMP FLOW OSCILLATION.

SUMMARY

ZION 2 OPERATED AT APPROXIMATELY 90% POWER FOR SEPTEMBER, WITH TWO OUTAGES ON THE 3RD AND 5TH FOR EQUIPMENT FAILURE.

LEGEND

[1] Type:
F-Forced
S-Scheduled

[2] Reason:
A-Equipment Failure
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error
H-Other

[3] Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Continuations
5-Load Reduction
9-Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

SECTION 3 – APPENDIX

STATUS OF SPENT FUEL STORAGE CAPABILITY

DATA AS OF 09/30/80

Facility	Core Size [a] (No. of Assemblies)	Present Authorized Storage Pool Capacity (Fuel Assemblies)	No. of Assemblies Stored	Remaining Capacity (No. of Assemblies)	Remaining Capacity If Pending Request Approved (No. of Assemblies)	Scheduled Date For Next Refueling	Will Fill Present [b] Authorized Capacity
I. Pressurized Water Reactors							
ARKANSAS 1	177	590	176	414		01-81	1986
ARKANSAS 2	177	486	58	428		03-81	1989
BEAVER VALLEY 1	157	833	0	833		N/S	1995
CALVERT CLIFFS 1	217	1056 (c)	364 (c)	692 (c)		10-80	1984
CALVERT CLIFFS 2	217					01-81	
COOK 1	193	500	193	307	1857	N/S	1985
COOK 2	193					N/S	
CRYSTAL RIVER 3	177	256	112	144	1109	N/S	1982
DAVIS-BESSE 1	177	735	52	683		03-82	1989
FARLEY 1	157	675	0	675		09-80	1990
FORT CALHOUN 1	133	483	197	286		03-81	1985
GINNA	121	595	156	439		N/S	1987
HADDAM NECK	157	1168	389	779		03-81	1994
INDIAN POINT 1	0	288	160	128		N/S	
INDIAN POINT 2	193	482	200	282		N/S	1984
INDIAN POINT 3	193	837	64	773		N/S	1989
KEWAUNEE	121	990	120	870		N/S	1999
MAINE YANKEE	217	953	433	520		N/S	1986
MILLSTONE 2	217	667	144	523		N/S	1987
NORTH ANNA 1	157	400	0	400	966	N/S	1984
NORTH ANNA 2 (d)	157	400	0	400			
OCONEE 1	177	1224 (l)	737	487 (l)	755	05-81	1986
OCONEE 2	177					06-81	
OCONEE 3	177					12-80	
PALISADES	204	798	273	525		N/S	1985
POINT BEACH 1	121	1502 (c)	276 (c)	1226 (c)		11-80	1995
POINT BEACH 2	121					03-81	
PRAIRIE ISLAND 1	121	687 (c)	280 (c)	407 (c)		09-80	1984
PRAIRIE ISLAND 2	121					01-81	
RANCHO SECO 1	177	579	164	415		04-81	1987
ROBINSON 2	157	272	106	166 (g)		N/S	1980 (g)
SALEM 1	193	264	40	224	1130	09-80	1982
SAN ONOFRE 1	157	216	94	119		N/S	1983
SEQUOYAH 1 (d)		0					
ST LUCIE 1	217	728	60	668		N/S	1990
SURRY 1	157	1044 (c)	436 (c)	608 (c)		09-80	1983
SURRY 2	157					09-80	
THREE MILE ISLAND 1	177	752	208	544		N/S	1986
THREE MILE ISLAND 2	177	442	0	442		N/S	1986
TROJAN	193	651	128	587		04-81	1986
TURKEY POINT 3	157	621	346 (c)	275 (c)		N/S	1987
TURKEY POINT 4	157					N/S	
YANKEE-ROWE 1	76	391	149	242	372	N/S	1985
ZION 1	193	2112 (c)	508 (c)	1604 (c)	1676	01-81	1993
ZION 2	193					N/S	

Facility	Core Size [a] (No. of Assemblies)	Present Authorized Storage Pool Capacity (Fuel Assemblies)	No. of Assemblies Stored	Remaining Capacity (No. of Assemblies)	Remaining Capacity If Pending Request Approved (No. of Assemblies)	Scheduled Date For Next Refueling	Will Fill Present [b] Authorized Capacity
II. Boiling Water Reactors							
BIG ROCK POINT 1	84	193	88	105	343	10-80	1983
BROWNS FERRY 1	764	3471	556	2795		04-81	1985
BROWNS FERRY 2	764	1080	640	440		09-80	1985
BROWNS FERRY 3	764	1528	516	1012		10-80	1985
BRUNSWICK 1	560	(f)	154PWR+476BWR			N/S	1986
BRUNSWICK 2	560		105PWR+264BWR			N/S	1986
COOPER STATION	548	2366	284	2082		N/S	1995
DRESDEN 1	464	672	221	451		N/S	1990
DRESDEN 2	724	2840 (c)	1069 (c)	1771 (c)	6491 (c)	12-80	1985
DRESDEN 3	724					N/S	
DUANE ARNOLD	368	2050	364	1686		04-81	1998
FITZPATRICK	560	760	428	492		09-81	1981
HATCH 1	560	840	260	580	1816	12-80	1986
HATCH 2	560	1120	0	1120		10-80	
HUMBOLDT BAY	172	487	251	236		N/S	
LA CROSSE	72	134	113	21		11-80	1990
MILLSTONE 1	580	2184	776	1408	327	09-80	1991
MONTICELLO	484	2237	828	1409		03-81	1991
NINE MILE POINT 1	532	1984	660	1324	2349	N/S	1990
OYSTER CREEK 1	560	1800	780	1020		N/S	1987
PEACH BOTTOM 2	764	2816	910	1906		01-82	1990
PEACH BOTTOM 3	764	2816	712	2104		03-81	1991
PILGRIM 1	580	2320	764	1556		09-81	1990
QUAD CITIES 1	724	1460	596	864		09-80	1986
QUAD CITIES 2	724	1460	492	968		N/S	1985
VERMONT YANKEE 1	368	2000	894	1106		09-80	1988
III. Independent Spent Fuel Storage Installations (h)							
Morris Operations		750 MTU (j)	315	386 MTU (j)	1490 MTU (j)		
NFS(i)		250 MTU	170 MTU	80 MTU			

(a) At each refueling outage approximately 1/3 of a PWR core and 1/4 of a BWR core is off-loaded.
(b) Some of these dates have been adjusted by staff assumptions.
(c) This is the total for both units.
(d) Plant not in commercial operation.
(e) Some spent fuel stored at Brunswick.
(f) Authorized a total 2088 BWR and 304 PWR assemblies for both pools.
(g) Robinson 2 assemblies being shipped to Brunswick for storage.
(h) Capacity is in metric tons of uranium; 1 MTU = 2 PWR assemblies or 5 BWR assemblies.
(i) No longer accepting spent fuel.
(j) Racked for 700 MTU.
(k) Reserved
(l) This is the station total.

N/S = Not Scheduled

best copy available

REACTOR-YEARS OF EXPERIENCE

09/30/80

The total of Reactor years of experience is as the sum of all calendar days for each unit, from the date that electricity was first generated until a final shutdown date or the status date, whichever comes first, divided by 365.25 days/year.

If a date is unknown, the first day of the first month of operation is substituted. Units which have not yet generated electricity but which are licensed are listed but not included in the computation.

LICENSED OPERATING ELECTRICAL PRODUCING UNITS

UNIT NAME	FIRST ELEC. GENERATION	YEARS EXPERIENCE
ARKANSAS 1	08/01/74	6.17
ARKANSAS 2	12/26/78	1.77
BEAVER VALLEY 1	06/14/76	4.30
BIG ROCK POINT 1	12/08/62	17.82
BROWNS FERRY 1	10/15/73	6.96
BROWNS FERRY 2	08/28/74	6.09
BROWNS FERRY 3	09/12/76	4.05
BRUNSWICK 1	12/04/76	3.82
BRUNSWICK 2	04/29/75	5.43
CALVERT CLIFFS 1	01/03/75	5.74
CALVERT CLIFFS 2	12/07/76	3.82
COOK 1	02/10/75	5.64
COOK 2	03/22/78	2.53
COOPER STATION	05/10/74	6.40
CRYSTAL RIVER 3	01/30/77	3.67
DAVIS-BESSE 1	08/28/77	3.09
DRESDEN 2	04/13/70	10.47
DRESDEN 3	07/22/71	9.20
DUANE ARNOLD	05/19/74	6.37
FARLEY 1	08/18/77	3.12
FITZPATRICK	02/01/75	5.66
FORT CALHOUN 1	08/25/73	7.10
FORT ST VRAIN	12/11/76	3.81
GINNA	12/02/69	10.83
HADDAM NECK	08/07/67	13.15
HATCH 1	11/11/74	5.89
HATCH 2	09/22/78	2.03
INDIAN POINT 2	06/26/73	7.27
INDIAN POINT 3	04/27/76	4.43
KEWAUNEE	04/08/74	6.48
LA CROSSE	04/26/68	12.43
MAINE YANKEE	11/08/72	7.90
MILLSTONE 1	11/29/70	9.84
MILLSTONE 2	11/09/75	4.90
MONTICELLO	03/05/71	9.58
NINE MILE POINT 1	11/09/69	10.89
NORTH ANNA 1	04/17/78	2.46
NORTH ANNA 2	08/25/80	.10
OCONEE 1	05/06/73	7.41
OCONEE 2	12/05/73	6.82
OCONEE 3	09/01/74	6.08
OYSTER CREEK 1	09/23/69	11.02
PALISADES	12/31/71	8.75
PEACH BOTTOM 2	02/18/74	6.62
PEACH BOTTOM 3	09/01/74	6.08
PILGRIM 1	07/19/72	8.20
POINT BEACH 1	11/06/70	9.90
POINT BEACH 2	08/02/72	8.16
PRAIRIE ISLAND 1	12/04/73	6.83
PRAIRIE ISLAND 2	12/21/74	5.78
QUAD CITIES 1	04/12/72	8.47
QUAD CITIES 2	05/23/72	8.36
RANCHO SECO 1	10/13/74	5.97
ROBINSON 2	09/26/70	10.02
SALEM 1	12/25/76	3.77

LICENSED OPERATING ELECTRICAL PRODUCING UNITS - CONT.

UNIT NAME	FIRST ELEC. GENERATION	YEARS EXPERIENCE
SAN ONOFRE 1	07/16/67	13.21
SEQUOYAH 1	07/22/80	.11
ST LUCIE 1	05/07/76	4.40
SURRY 1	07/04/72	8.24
SURRY 2	03/10/73	7.56
THREE MILE ISLAND 1	06/19/74	6.29
TROJAN	12/23/75	4.77
TURKEY POINT 3	11/02/72	7.91
TURKEY POINT 4	06/21/73	7.28
VERMONT YANKEE 1	09/20/72	8.03
YANKEE-ROWE 1	11/10/60	19.89
ZION 1	06/28/73	7.26
ZION 2	12/26/73	6.77
TOTAL		471.18 YEARS

PERMANENTLY OR INDEFINITELY SHUTDOWN UNITS

UNIT NAME	FIRST ELEC. GENERATION	SHUTDOWN OR ELECTRICAL CESSATION DATE	YEARS EXPERIENCE
BONUS	08/14/64	06/01/68	3.80
CVTR	12/18/63	01/01/67	3.04
DRESDEN 1	04/15/60	10/31/78	18.54
ELK RIVER	08/24/63	02/01/68	4.44
FERMI 1	08/05/66	11/29/72	6.32
HALLAM	05/29/63	09/01/64	1.26
HUMBOLDT BAY	04/18/63	07/02/76	13.21
INDIAN POINT 1	09/16/62	10/31/74	12.12
PATHFINDER	07/25/66	10/01/67	1.19
PEACH BOTTOM 1	01/27/67	11/01/74	7.76
PIQUA	11/04/63	01/01/66	2.16
THREE MILE ISLAND 2	04/21/78	03/28/79	.93
TOTAL			74.77 YEARS

NON–POWER REACTORS IN THE U.S.

1. RESEARCH REACTORS

State	Location	Licensee	Reactor Type	Docket	License Number	Date Issued	Authorized Power Level (KW)
Alabama	1. Tuskegee	Tuskegee Institute	AGN 201 No. 102	50-406	R-122	08-30-74	.0001
Arizona	2. Tucson	U. of Arizona	TRIGA Mk. I	50-113	R-52	12-05-58	100
California	3. Los Angeles	U. of California	Argonaut	50-142	R-71	10-03-60	100
	4. Berkeley	U. of California	TRIGA Mark III	50-224	R-101	08-10-66	1000
	5. Irvine	U. of California	TRIGA Mark I	50-326	R-116	11-24-69	250
	6. Santa Barbara	U. of California	L-77	50-433	R-124	12-03-74	.01
	7. Canoga Park	Rockwell International Corporation	L-77	50-94	R-40	05-17-58	.01
	8. Hawthorne	Northrop Corporate Laboratories	TRIGA Mark F	50-187	R-90	03-04-63	1000
	9. San Luis Obispo	California State Polytechnic College	AGN 201 No. 100	50-394	R-121	05-16-73	.0001
	10. Alameda County	General Electric Co.	NTR	50-73	R-33	10-31-57	100
	11. San Diego	General Atomic Co.	TRIGA Mark I	50-89	R-38	05-03-58	250
	12. San Diego	General Atomic Co.	TRIGA Mark F	50-163	R-67	07-01-60	1500
	13. San Ramon	Aerotest Operations, Inc.	TRIGA (Industrial)	50-228	R-98	07-02-65	250
Colorado	14. Jefferson County	U.S. Geological Survey. Dept. of Interior	TRIGA Mark I	50-274	R-113	02-24-69	1000
Delaware	15. Newark	U. of Delaware	AGN-201 No. 113	50-98	R-43	07-03-58	.0001
District of Columbia	16. —	Catholic U.	AGN-201 No. 101	50-77	R-31	11-15-67	.0001
Florida	17. Gainesville	U. of Florida	Argonaut	50-83	R-56	05-21-59	100
Georgia	18. Atlanta	Georgia Institute of Technology	AGN-201 No. 104	50-276	R-111	04-19-68	.0001
	19. Atlanta	Georgia Institute of Technology	Heavy Water	50-160	R-97	12-29-64	5000
Idaho	20. Pocatello	Idaho State U.	AGN-201 No. 103	50-284	R-110	10-11-67	.0001
Illinois	21. Urbana	U. of Illinois	Advanced TRIGA	50-151	R-115	07-22-69	1500
	22. Urbana	U. of Illinois	LOPRA	50-356	R-117	12-27-71	10
	23. Zion	Westinghouse Electric Corporation	NTR	50-87	R-119	01-28-72	10
Indiana	24. West Lafayette	Purdue U.	Lockheed	50-182	R-87	08-16-62	10

RESEARCH REACTORS – CONT.

State	Location	Licensee	Reactor Type	Docket	License Number	Date Issued	Authorized Power Level (KW)
Iowa	25. Ames	Iowa State U.	UTR-10	50-116	R-59	10-16-59	10
Kansas	26. Lawrence	U. of Kansas	Lockheed	50-148	R-78	06-23-61	250
	27. Manhattan	Kansas State U.	TRIGA	50-188	R-88	10-16-62	250
Maryland	28. College Park	U. of Maryland	TRIGA	50-166	R-70	10-14-60	250
	29. Bethesda	Armed Forces Radiobiology Research Institute	TRIGA	50-170	R-84	06-26-62	1000
Massachusetts	30. Lowell	U. of Lowell	GE	50-223	R-125	12-24-74	1000
	31. Cambridge	Massachusetts Institute of Technology	Heavy Water Reflected	50-20	R-37	06-09-58	5000
	32. Worcester	Worcester Polytechnic Institute	GE	50-134	R-61	12-16-59	10
Michigan	33. Ann Arbor	U. of Michigan	Pool	50-2	R-28	09-13-57	2000
	34. Lansing	Michigan State U.	TRIGA Mark I	50-294	R-114	03-21-69	250
	35. Midland	Dow Chemical Co.	TRIGA Mark I	50-264	R-108	07-03-67	100
Missouri	36. Rolla	U. of Missouri	Pool	50-123	R-79	11-21-61	200
	37. Columbia	U. of Missouri	Tank	50-186	R-103	10-11-66	10,000
Nebraska	38. Omaha	V.A. Hospital	TRIGA	50-131	R-57	06-26-59	18
New Mexico	39. Albuquerque	U. of New Mexico	AGN-201M No. 112	50-252	R-102	09-17-66	.005
New York	40. Buffalo	State U. of New York	PULSTAR	50-57	R-77	03-24-61	2000
	41. Sterling Forest	Union Carbide Corp.	Pool	50-54	R-81	09-07-61	5000
	42. New York City	Columbia U.	TRIGA Mark II	50-208	R-128	04-14-77	250
	43. Ithaca	Cornell U.	ZPR	50-97	R-89	12-11-62	.1
	44. Ithaca	Cornell U.	TRIGA Mk. II	50-157	R-80	01-11-62	100
	45. Riverdale	Manhattan College	Tank	50-199	R-94	03-24-64	.0001
North Carolina	46. Raleigh	North Carolina State U.	PULSTAR	50-297	R-120	08-25-72	1000
Ohio	47. Columbus	Ohio State U.	Pool	50-150	R-75	02-24-61	10
Oklahoma	48. Norman	U. of Oklahoma	AGN-211 No. 102	50-112	R-53	12-29-58	.015
Oregon	49. Corvallis	Oregon State U.	AGN-201 No. 114	50-106	R-51	11-13-58	.0001
	50. Corvallis	Oregon State U.	TRIGA Mark II	50-243	R-106	03-07-67	1000
	51. Portland	Reed Institute	TRIGA Mk. I	50-288	R-112	07-02-68	250
Pennsylvania	52. University Park	Pennsylvania State U.	TRIGA Mk. III	50-5	R-2	07-08-55	1000

NON–POWER REACTORS IN THE U.S.

RESEARCH REACTORS – CONT.

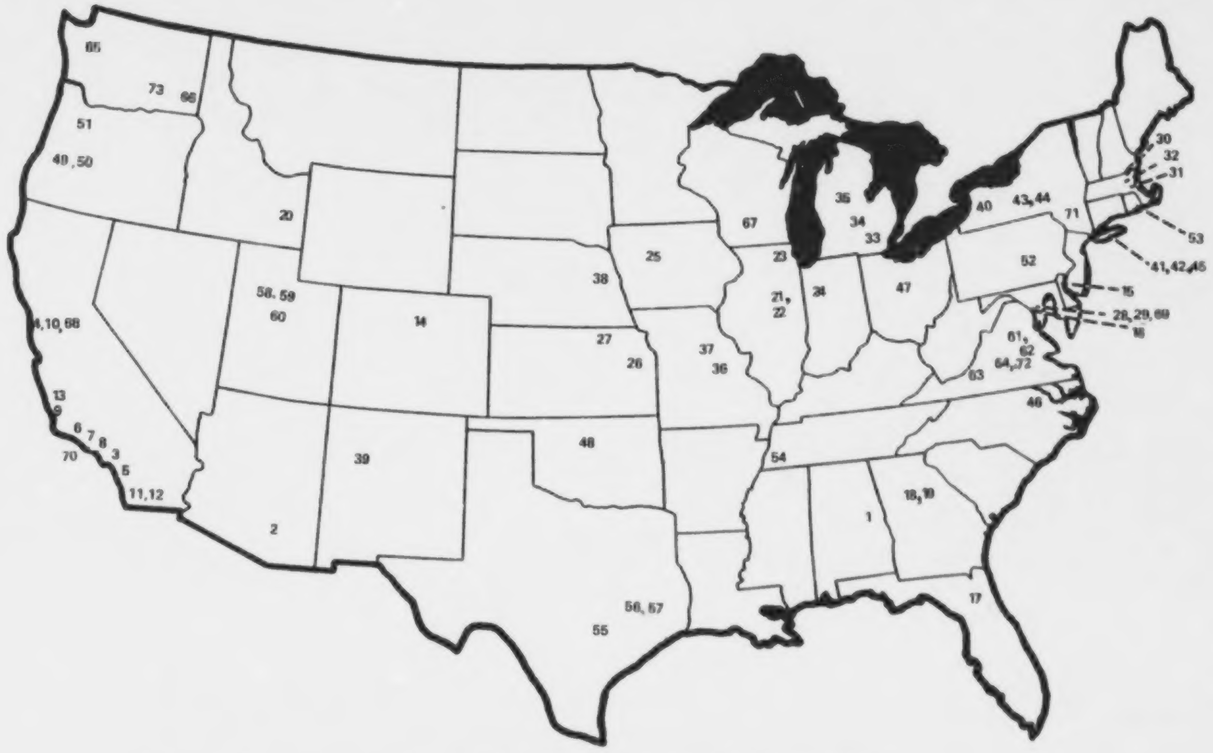
State	Location	Licensee	Reactor Type	Docket	License Number	Date Issued	Authorized Power Level (KW)
Rhode Island	53. Ft. Kearney, Narragansett	Rhode Island & Providence Plantations AEC	GE Pool	50-193	R-95	07-21-64	2000
Tennessee	54. Shelby County	Memphis State U.	AGN-201 No. 108	50-538	R-127	12-10-76	.0001
Texas	55. Austin	U. of Texas	TRIGA Mk. I	50-192	R-92	08-02-63	250
	56. College Station	Texas A&M U.	TRIGA	50-128	R-83	12-07-61	1000
	57. College Station	Texas A&M U.	AGN- 201M No. 106	50-59	R-23	08-26-57	.005
Utah	58. Salt Lake City	U. of Utah	AGN-201M No. 107	50-72	R-25	09-12-57	.005
	59. Salt Lake City	U. of Utah	TRIGA Mark I	50-407	R-126	09-30-75	100
	60. Provo	Brigham Young U.	L-77	50-262	R-109	09-07-67	.01
Virgina	61. Charlottesville	U. of Virgina	Pool	50-62	R-66	06-27-60	2000
	62. Charlottesville	U. of Virginia	Cavalier	50-396	R-123	09-24-74	.1
	63. Blacksburg	Virginia Polytechnic Institute	UTR-10	50-124	R-62	12-18-59	100
	64. Lynchburg	Babcock & Wilcox Co.	LPR	50-99	R-47	09-05-58	1000
Washington	65. Seattle	U. of Washington	Argonaut	50-139	R-73	03-31-61	100
	66. Pullman	Washington State U.	TRIGA	50-27	R-76	03-06-61	1000
Wisconsin	67. Madison	U. of Wisconsin	TRIGA	50-156	R-74	11-23-60	1000

2. EXPERIMENTAL AND TEST REACTORS

State	Location	Licensee	Reactor Type	Docket	License Number	Date Issued	Authorized Power Level (MWt)
California	68. Alameda Co.	General Electric Co.	GETR	50-70	TR-1	01-07-59	50
Maryland	69. Gaithersburg	National Bureau of Standards	TEST	50-184	TR-5	11-13-67 POL 06-30-70 OL	10

3. CRITICAL EXPERIMENT FACILITIES

State	Location	Licensee	Docket	License Number	Date Issued	Experiments
California	70. Ventura Co.	Rockwell International Corp. (Possesion Only)	50-147	CX-17	10-03-60	AETR Experiments (Terminal Requested).
New York	71. Schenectady	Rensselaer Polytechnic Institute	50-225	CX-22	07-03-64	
Virginia	72. Lynchburg	Babcock & Wilcox Co.	50-13	CX-10	10-22-58	NMSR Experiments
Washington	73. Richland	Battelle Memorial Institute	50-300	CX-26	11-29-71	Plutonium Recycle Critical Facility



Source: NRC Facilities License Application Record (FLAR) 06-30-77.

ABBREVIATIONS

AGN	Aerojet General Nuclear	These reactors were constructed by Aeroject at San Ramon, California, and then transferred to the current location.
ZPR	Zero Power Reactor	
LPR	Low Power Reactor	

DISTRIBUTION LIST—NRC

INTERNAL DISTRIBUTION

Office of the Commissioners	5
Atomic Safety and Licensing Appeal Panel	3
Advisory Committee on Reactor Safeguards	6
Office of Inspector and Auditor	1
Office of Policy Evaluation	1
Office of the General Counsel	1
Office of Public Affairs	21
Office of Congressional Affairs	3
Office of the Executive Director for Operations	7
Office of Administration	2
Office of the Executive Legal Director	2
Office of the Controller	2
Office for Analysis and Evaluation of Operational Data	2
Office of International Programs	3
Office of State Programs	1
Office of Management and Program Analysis	14
Office of Standards Development	1
Office of Nuclear Material Safety and Safeguards	2
— Division of Fuel Cycle and Material Safety	3
— Division of Safeguards	2
Office of Nuclear Reactor Regulation Division of Licensing Division of Engineering Division of Systems Integration Division of Human Factors Safety Division of Safety Technology	138
Office of Nuclear Regulatory Research	4
Office of Inspection and Enforcement	7
— Region I	11
— Region II	10
— Region III	10
— Region IV	10
— Region V	7
	279

EXTERNAL DISTRIBUTION

SPECIAL REQUESTS

Air Force	1
Congress	10
Department of Energy	32
Government Accounting Office	1
Environmental Protection Agency	1
Electric Power Research Institute	1
Argonne National Laboratory	3
Federal Power Commission	1
Bureau of Mines	1
Department of Agriculture	1
Department of Commerce	1
Department of the Interior	1
	54

OTHER

GPO Depository	440
GPO Stores	75
National Technical Information Service	25
Subscriptions (NTIS)	250
Colleges and Libraries (including Public Document Rooms)	128
Utilities and Other Requests	227
	1145

15/4-11

Nuclear Regulatory Commission

NUREG 0020
VOL. 4 NO. 11

OPERATING UNITS STATUS REPORT

NOVEMBER 1980

DATA AS OF 10/31/80

NEWARK PUBLIC LIBRARY
JAN 14 1981
U. S. GOVT. PUBS.
DEPOSITORY 376

LICENSED OPERATING REACTORS

Y3.N88:

Available from
GPO Sales Program
Division of Technical Information and Document Control
U.S. Nuclear Regulatory Commission
and
National Technical Information Service
Springfield, Virginia 22161
(703) 326-8543
A Year's Subscription, 12 Issues, for this Publication is
$120.00 North American Continent (240.00 Other).
Multiple Subscriptions Entered at the Same Time to the Same Address
Cost $90.00 North American Continent ($180.00 Other) for each Additional Subscription.

Single copies are $12.50 North American Continent ($25.00 Other).

This document is provided free of charge to those who participate in the data collection program, and to the Nuclear Regulatory Commission's local public document rooms.

Issued by the U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

PUBLISHED: DECEMBER 15, 1980

Nuclear Regulatory Commission

NUREG 0020
VOL. 4 NO. 11

OPERATING UNITS STATUS REPORT

NOVEMBER 1980

DATA AS OF 10/31/80

LICENSED OPERATING REACTORS

AUTHORIZATION AND CLEARANCE

THE U.S. NUCLEAR REGULATORY COMMISSION'S OFFICE OF MANAGEMENT AND PROGRAM ANALYSIS PUBLISHES THIS MONTHLY STATUS REPORT "AS PART OF THE REPORTING REQUIREMENTS IN SECTION 50.36 OF 10 CFR PART 50 UNDER GAO CLEARANCE NUMBER B-180225, WITH AN EXPIRATION DATE OF SEPTEMBER 30, 1981," AS STATED IN THE OCTOBER 3, 1978 LETTER FROM JOHN M. LOVELADY, ASSISTANT DIRECTOR, GENERAL GOVERNMENT DIVISION, U. S. GENERAL ACCOUNTING OFFICE, TO J.M. FELTON, DIRECTOR, DIVISION OF RULES AND RECORDS, U.S. NUCLEAR REGULATORY COMMISSION.

STATEMENT OF PURPOSE

The U. S. Nuclear Regulatory Commission's monthly OPERATING UNITS STATUS REPORT — LICENSED OPERATING REACTORS, provides data on the operation of nuclear units as timely and accurately as possible. This information is collected by the Office of Management and Program Analysis from the Headquarters Staff of NRC's Office of Inspection and Enforcement, from NRC's Regional Offices, and from utilities. Since all of the data concerning operation of the units is provided by the utility operators less than two weeks after the end of the month, necessary corrections to published information are shown on the ERRATA page.

This report is divided into three sections: the first contains monthly highlights and statistics for commercial operating units, and errata from previously reported data; the second is a compilation of detailed information on each unit, provided by NRC Regional Offices, IE Headquarters and the Utilities; and the third section is an appendix for miscellaneous information such as spent fuel storage capability, reactor-years of experience and non-power reactors in the United States.

The percentage computations, Items 19 through 24 in Section 2, the vendor capacity factors on page 1-4, and actual vs. potential energy production on Page 1-2 are computed from actual power production from the period of consideration. The percentages listed in power generation on Page 1-2 are computed as an arithmetical average. The factors for the life-span of each unit (the "Cumulative" column) are reported by the utility and are not entirely re-computed by NRC. Utility power production data is checked for consistency with previously submitted statistics.

In some cases, the numbers in the "Cumulative" column may have been totaled from a date that is different from the date of commercial operation. In these cases, the column is noted with an explanation of the nature of the change on each individual plant page in Section 2.

It is hoped this status report proves informative and helpful to all agencies and individuals interested in analyzing trends in the nuclear industry which might have safety implications, or in maintaining an awareness of the U. S. energy situation as a whole.

TABLE OF CONTENTS

GLOSSARY ii
INDEX TO LICENSED UNITS iii

SECTION 1 – CURRENT DATA SUMMARIES

MONTHLY HIGHLIGHTS OF COMMERCIAL NUCLEAR POWER UNITS 1 2

Licensed Power Reactors 1-2
Power Generation 1-2
Actual vs. Potential Energy Production 1-2
Outage Data 1-2
Derated Units 1-2
Shutdowns - Number and Reasons 1-3
Unit Availability, Capacity, and Forced Outage Rate 1-3
Refueling Schedule 1-3

AVERAGE CAPACITY FACTORS BY VENDOR 1-4

AVERAGE DAILY POWER LEVEL FOR ALL COMMERCIAL OPERATING UNITS 1-5

ERRATA 1-7

SECTION 2 – OPERATING POWER REACTORS

ARKANSAS 1 THROUGH ZION 2 PAGES A–2 TO Z–5

Facility Data
Inspection and Enforcement Status
Reports Received from Licensee
Operating Status
Average Daily Power Level (MWe)
Unit Shutdowns/Reductions

SECTION 3 – APPENDIX

STATUS OF SPENT FUEL STORAGE CAPABILITY 3-2
REACTOR-YEARS OF EXPERIENCE 3-3
NON-POWER REACTORS IN THE U.S. 3-4

GLOSSARY

AVERAGE DAILY POWER LEVEL (MWe)	The net electrical energy generated during the day (measured from 0001 to 2400 hours inclusive) in megawatts hours divided by 24 hours.
LICENSED THERMAL POWER (MWt)	The maximum thermal power of the reactor authorized by the NRC, expressed in megawatts.
DATE OF COMMERCIAL OPERATION	Date unit was declared by utility owner to be available for the regular production of electricity; usually related to satisfactory completion of qualification tests as specified in the purchase contract and to accounting policies and practices of utility.
DESIGN ELECTRICAL RATING (DER) (Net MWe)	The nominal net electrical output of the unit specified by the utility and used for the purpose of plant design.
FORCED OUTAGE	An outage required to be initiated no later than the weekend following discovery of an off-normal condition.
FORCED OUTAGE HOURS	The clock hours during the report period that a unit is unavailable due to forced outages.
GROSS ELECTRICAL ENERGY GENERATED (MWH)	Electrical output of the unit during the report period as measured at the output terminals of the turbine generator, in megawatts hours.
GROSS HOURS	The clock hours from the beginning of a specified situation until its end. For outage durations, the clock hours during which the unit is not in power production.
GROSS THERMAL ENERGY GENERATED (MWH)	The thermal energy produced by the unit during the report period as measured or computed by the licensee in megawatt hours.
HOURS GENERATOR ON-LINE	Also, "Unit Service Hours." The total clock hours in the report period during which the unit operated with breakers closed to the station bus. These hours added to the total outage hours experienced by the unit during the report period, shall equal the hours in the report period.
HOURS IN REPORTING PERIOD	For units in power ascension at the end of the period, the gross hours from the beginning of the period or the first electrical production, whichever comes last, to the end of the period. For units in commercial operation at the end of the period, the gross hours from the beginning of the period or of commercial operation, whichever comes last, to the end of the period or decommissioning, whichever comes first.
HOURS REACTOR CRITICAL	The total clock hours in the report period during which the reactor sustained a controlled chain reaction.
MAXIMUM DEPENDABLE CAPACITY (GROSS) (MDC Gross) (Gross MWe)	Dependable main-unit gross capacity, winter or summer, whichever is smaller. The dependable capacity varies because the unit efficiency varies during the year due to cooling water temperature variations. It is the gross electrical output as measured at the output terminals of the turbine generator during the most restrictive seasonal conditions (usually summer).
MAXIMUM DEPENDABLE CAPACITY (NET) (MDC Net) (Net MWe)	Maximum Dependable Capacity (Gross) less the normal station service loads.
NAMEPLATE RATING (Gross MWe)	The nameplate power designation of the generator in megavolt amperes (MVA) times the nameplate power factor of the generator. NOTE: The nameplate rating of the generator may not be indicative of the maximum or dependable capacity, since some other item of equipment of a lesser rating (e.g., turbine) may limit unit output.
NET ELECTRICAL ENERGY GENERATED	Gross electrical output of the unit measured at the output terminals of the turbine generator during the reporting period, minus the normal station service electrical energy utilization. If this quantity is less than zero, a negative number should be recorded.
OUTAGE	A situation in which no electrical production takes place.
OUTAGE DATE	As reported on Appendix D of Reg. Guide 1.16, the date of the start of the outage. If continued from a previous month, report the same outage date but change "Method of Shutting Down Reactor" to "4 (Other)" and add a note: "Continued from previous month."
OUTAGE DURATION	The total clock hours of the outage measured from the beginning of the report period or the outage, whichever comes last, to the end of the report period or the outage, whichever comes first.
OUTAGE NUMBER	A number unique to the outage assigned by the licensee. The same number is reported each month in which the outage is in progress. One format is "76-05" for the fifth outage to occur in 1976.
PERIOD HOURS	See "Hours in Reporting Period."
POWER REDUCTION	A reduction in the Average Daily Power Level of more than 20% from the previous day. All power reductions are defined as outage of zero hours durations for the purpose of computing unit service and availability factors, and forced outage rate.

REACTOR AVAILABLE HOURS	The total clock hours in the report period during which the reactor was critical or was capable of being made critical. (Reactor Reserve Shutdown Hours + Hours Reactor Critical.)
REACTOR AVAILABILITY FACTOR	$\frac{\text{Reactor Available Hours} \times 100}{\text{Period Hours}}$
REACTOR RESERVE SHUTDOWN	The cessation of criticality in the reactor for administrative or other similar reasons when operation could have been continued.
REACTOR RESERVE SHUTDOWN HOURS	The total clock hours in the report period during which the reactor is in reserve shutdown mode. NOTE: No credit is given for NRC imposed shutdowns.
REACTOR SERVICE FACTOR	$\frac{\text{Hours Reactor Critical} \times 100}{\text{Period Hours}}$
REPORT PERIOD	Usually, the preceding calendar month. Can also be the preceding calendar year, (Year-to-Date), or the life-span of a unit (cumulative).
RESTRICTED POWER LEVEL	Maximum net electrical generation to which the unit is restricted during the report period due to the state of equipment, external conditions, administrative reasons, or a direction by NRC.
SCHEDULED OUTAGE	Planned removal of a unit from service for refueling, inspection, training, or maintenance. Those outages which do not fit the definition of "Forced Outage" perforce are "Scheduled Outages."
STARTUP AND POWER ASCENSION TEST PHASE	Period following initial criticality during which the unit is tested at successively higher levels, culminating with operation at full power for a sustained period and completion of warranty runs. Following this phase, the utility generally considers the unit to be available for commercial operation.
UNIT	The set of equipment uniquely associated with the reactor, including turbine generators, and ancillary equipment, considered as a single electrical energy production facility.
UNIT AVAILABLE HOURS	The total clock hours in the report period during which the unit operated on-line or was capable of such operation. (Unit Reserve Shutdown Hours + Hours Generator On-Line.)
UNIT AVAILABILITY FACTOR*	$\frac{\text{Unit Available Hours} \times 100}{\text{Period Hours}}$
UNIT CAPACITY FACTORS	
– Using Licensed Thermal Power	$\frac{\text{Gross Thermal Energy Generated} \times 100}{\text{Period Hours} \times \text{Lic. Thermal Power}}$
– Using Nameplate Rating	$\frac{\text{Gross Electrical Energy Generated} \times 100}{\text{Period Hours} \times \text{Nameplate Rating}}$
– Using DER	$\frac{\text{Net Electrical Energy Generated} \times 100}{\text{Period Hours} \times \text{DER}}$
– Using MDC Gross	$\frac{\text{Gross Electrical Energy Generated} \times 100}{\text{Period Hours} \times \text{MDC Gross}}$
– Using MDC Net	$\frac{\text{Net Electrical Energy Generated} \times 100}{\text{Period Hours} \times \text{MDC Net}}$

NOTE: if MDC Gross and/or MDC Net have not been determined, the DER is substituted for this quantity for Unit Capacity Factor calculations.

UNIT FORCED OUTAGE RATE	$\frac{\text{Forced Outage Hours}}{\text{Unit Service Hours} + \text{Forced Outage Hours}}$
UNIT RESERVE SHUTDOWN	The removal of the unit from on-line operation for economic or other similar reasons when operation could have been continued.
UNIT RESERVE SHUTDOWN HOURS	The total clock hours in the report period during which the unit was in reserve shutdown mode.
UNIT SERVICE FACTOR	$\frac{\text{Unit Service Hours} \times 100}{\text{Period Hours}}$
UNIT SERVICE HOURS	See "Hours Generator On-Line."

INDEX TO LICENSED UNITS

ARKANSAS 1 A-3
ARKANSAS 2 A-5
BEAVER VALLEY 1 B-3
BIG ROCK POINT 1 B-5
BROWNS FERRY 1 B-7
BROWNS FERRY 2 B-9
BROWNS FERRY 3 B-11
BRUNSWICK 1 B-13
BRUNSWICK 2 B-15
CALVERT CLIFFS 1 C-3
CALVERT CLIFFS 2 C-5
COOK 1 C-7
COOK 2 C-9
COOPER STATION C-11
CRYSTAL RIVER 3 C-13
DAVIS-BESSE 1 D-3
DRESDEN 1 D-5
DRESDEN 2 D-7
DRESDEN 3 D-9
DUANE ARNOLD D-11
FARLEY 1 F-3
FITZPATRICK F-5
FORT CALHOUN 1 F-7
FORT ST VRAIN F-9
GINNA G-3
HADDAM NECK H-3
HATCH 1 H-5
HATCH 2 H-7
HUMBOLDT BAY H-9
INDIAN POINT 2 I-3
INDIAN POINT 3 I-5
KEWAUNEE K-3
LA CROSSE L-3
MAINE YANKEE M-3
MILLSTONE 1 M-5
MILLSTONE 2 M-7
MONTICELLO M-9
NINE MILE POINT 1 N-3
NORTH ANNA 1 N-5
NORTH ANNA 2 N-7
OCONEE 1 O-3
OCONEE 2 O-5
OCONEE 3 O-7
OYSTER CREEK 1 O-9
PALISADES P-3
PEACH BOTTOM 2 P-5
PEACH BOTTOM 3 P-7
PILGRIM 1 P-9
POINT BEACH 1 P-11
POINT BEACH 2 P-13
PRAIRIE ISLAND 1 P-15
PRAIRIE ISLAND 2 P-17
QUAD CITIES 1 Q-3
QUAD CITIES 2 Q-5
RANCHO SECO 1 R-3
ROBINSON 2 R-5
SALEM 1 S-3
SAN ONOFRE 1 S-5
SEQUOYAH 1 S-7
ST LUCIE 1 S-9
SURRY 1 S-11
SURRY 2 S-13
THREE MILE ISLAND 1 T-3
THREE MILE ISLAND 2 T-5
TROJAN T-7
TURKEY POINT 3 T-9
TURKEY POINT 4 T-11
VERMONT YANKEE 1 V-3
YANKEE-ROWE 1 Y-3
ZION 1 Z-3
ZION 2 Z-5

SECTION 1 — CURRENT DATA SUMMARIES

MONTHLY HIGHLIGHTS OF COMMERCIAL NUCLEAR POWER UNITS

FOR MONTH OF

OCTOBER 1980

LICENSED POWER REACTORS:

NUMBER		CAPACITY [1] MWe (Net)
66	IN COMMERCIAL OPERATION	48,711
2	IN POWER ASCENSION PHASE	2,046
68 [2]	LICENSED TO OPERATE	50,757
2 [3]	LICENSED FOR FUEL LOADING AND LOW POWER TESTING	

POWER GENERATION	REPORT MONTH	PREVIOUS MONTH	YEAR-TO-DATE
1. GROSS ELECTRICAL (MWhe)	25,177,096	24,544,195	216,627,167
2. NET ELECTRICAL (MWhe)	24,052,535	23,356,389	206,221,508
3. AVG. UNIT SERVICE FACTOR (%)	72.2	74.1	65.2
4. AVERAGE UNIT AVAILABILITY FACTOR (%)	72.5	74.1	65.3
5. AVERAGE UNIT CAPACITY FACTOR (MDC) (%	65.5	65.5	57.5
6. AVG. UNIT CAPACITY FACTOR (DER) (%)	63.4	63.5	56.0
7. FORCED OUTAGE RATE (%)	8.1	11.0	11.2

NOTES

[1] BASED UPON MAXIMUM DEPENDABLE CAPACITY; DESIGN ELECTRIC RATING USED IF MDC NOT DETERMINED.

[2] EXCLUDES DRESDEN 1 (DER = 200), HUMBOLDT BAY (DER = 65), AND THREE MILE ISLAND 2 (DER = 906) WHICH HAVE OPERATING LICENSES BUT ARE SHUTDOWN INDEFINITELY.

[3] SALEM 2 (DER = 1115) WAS LICENSED ON APRIL 18, 1980
FARLEY 2 (DER=829) WAS LICENSED ON OCTOBER 23, 1980.

ACTUAL VS. POTENTIAL ENERGY PRODUCTION

			% OF POTENTIAL PRODUCTION
POTENTIAL ENERGY PRODUCTION IN THIS PERIOD BY UNITS IN COMMERCIAL OPERATION (Using Maximum Dependable Capacity Net)		36,289,695 MWHe	
ENERGY ACTUALLY PRODUCED DURING THIS REPORT PERIOD	=	24,052,535 MWHe NET	66.3
ENERGY NOT PRODUCED DURING THIS REPORT PERIOD DUE TO SCHEDULED OUTAGES (NET)	=	7,690,438 MWHe	21.2
ENERGY NOT PRODUCED DURING THIS REPORT PERIOD DUE TO FORCED OUTAGES (NET)	=	2,384,372 MWHe	6.6
ENERGY NOT PRODUCED DURING THIS REPORT PERIOD FOR OTHER REASONS (NET)	=	2,162,351 MWHe	6.0
TOTAL		36,289,695 MWHe =	100%
ENERGY NOT PRODUCED THIS REPORT PERIOD DUE TO NRC IMPOSED SHUTDOWNS AND RESTRICTIONS	=	1,204,630 MWHe	3.3
NUMBER OF UNITS WITH POWER RESTRICTIONS IMPOSED BY NRC	=	1	

OUTAGE DATA

	NUMBER	HOURS	PERCENT OF CLOCK TIME	MWHE LOST PRODUCTION
1. FORCED OUTAGES DURING MONTH	52	3,917.5	8.0	2,384,372
2. SCHEDULED OUTAGES DURING THE MONTH	22	9,745.0	19.8	7,690,438
3. TOTAL	74	13,662.5	27.8	10,074,809

MWHE LOST PRODUCTION = Down time x maximum dependable capacity net

DERATED UNITS

UNIT NAME	MAXIMUM DEPENDABLE CAPACITY (MWe - NET)	POWER LIMIT (MWe - NET)
†FORT ST VRAIN	330	231
INDIAN POINT 2	849	824
INDIAN POINT 3	965	917
NINE MILE POINT 1	610	579
OYSTER CREEK 1	620	620
POINT BEACH 1	495	390

†Restricted by NRC

MONTHLY HIGHLIGHTS OF COMMERCIAL NUCLEAR POWER UNITS

FOR MONTH OF

OCTOBER 1980

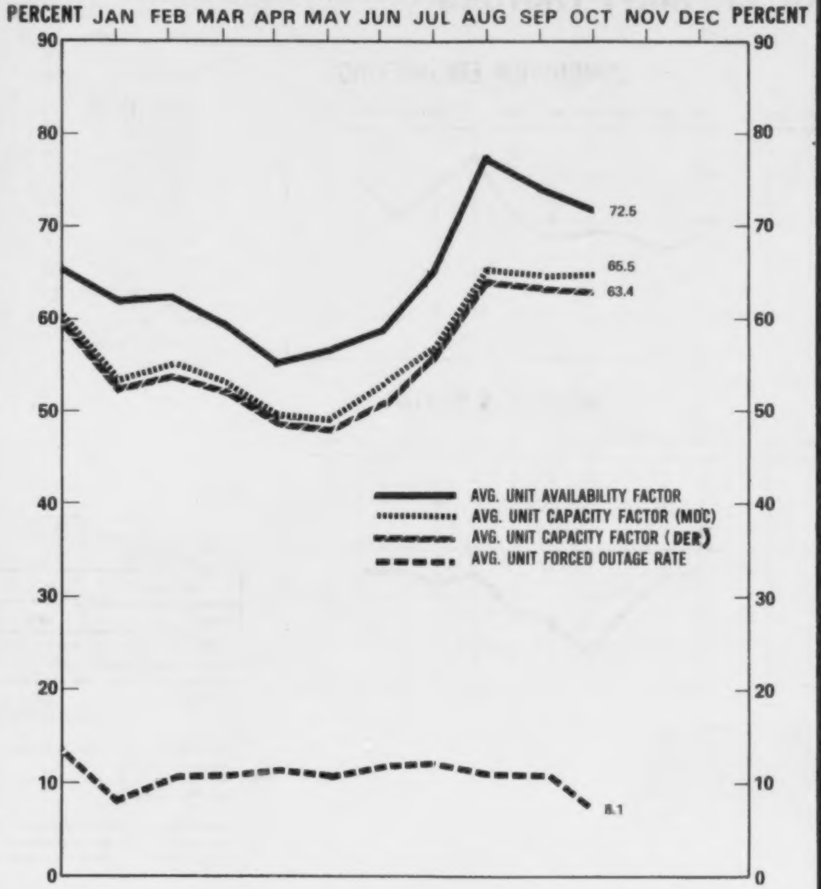
REASONS FOR SHUTDOWN

REASON	NUMBER	TOTAL HOURS LOST
A - Equipment Failure	35	2,906.0
B - Maintenance or Test	15	2,567.4
C - Refueling	11	6,314.3
D - NRC Restriction	2	1,490.0
E - Operator Training and License Examination	0	0
F - Administrative	0	0
G - Operational Error	7	150.5
H - Other	4	234.3
TOTAL	74	13,662.5

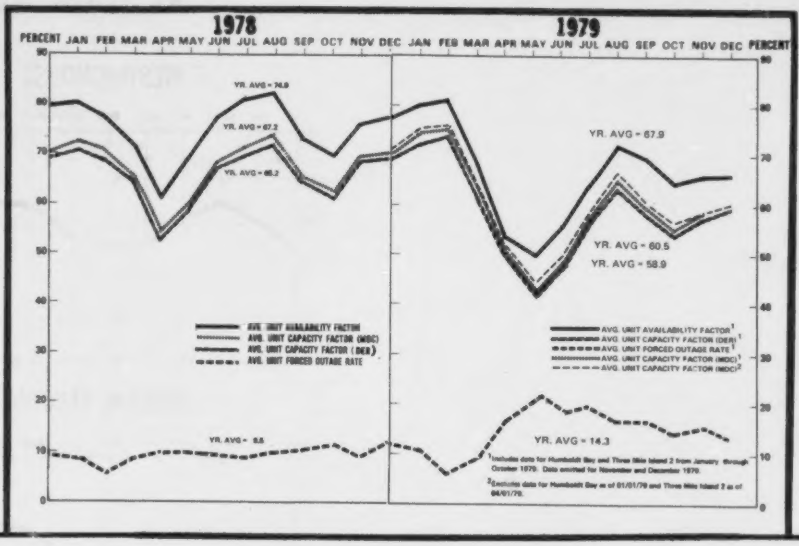
SHUTDOWNS GREATER THAN 72 HOURS EACH

UNIT NAME	REASON	UNIT NAME	REASON
BEAVER VALLEY 1	D	PILGRIM 1	A
BROWNS FERRY 2	C	PRAIRIE ISLAND 1	C
CALVERT CLIFFS 1	C	QUAD CITIES 1	C
COOK 2	A	QUAD CITIES 2	B
DAVIS-BESSE 1	C	ROBINSON 2	C
DRESDEN 2	A	SALEM 1	C
FITZPATRICK	A	SAN ONOFRE 1	B
FORT ST VRAIN	A,H	SURRY 1	A
INDIAN POINT 2	A	THREE MILE ISLAND 1	D
INDIAN POINT 3	B	TURKEY POINT 3	B
MILLSTONE 1	C	VERMONT YANKEE 1	C
MILLSTONE 2	C	YANKEE-ROWE 1	A
PEACH BOTTOM 3	B		

UNIT AVAILABILITY, CAPACITY AND FORCED OUTAGE



UNIT AVAILABILITY, CAPACITY AND FORCED OUTAGE



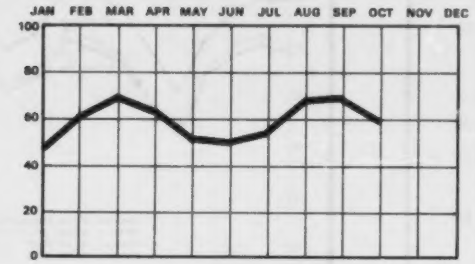
REFUELING SCHEDULE

Please refer to the table on page 3-2 for the scheduled refueling date for each nuclear powerplant in this publication.

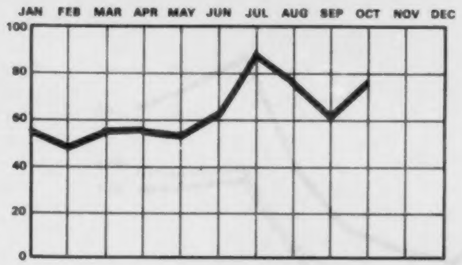
VENDOR AVERAGE CAPACITY FACTORS

OCTOBER 1980
OPERATIONS

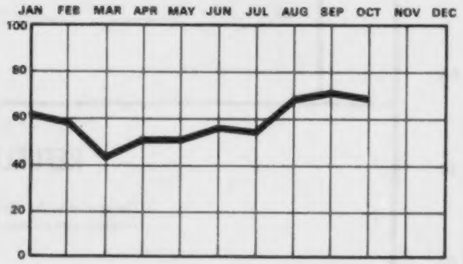
WESTINGHOUSE



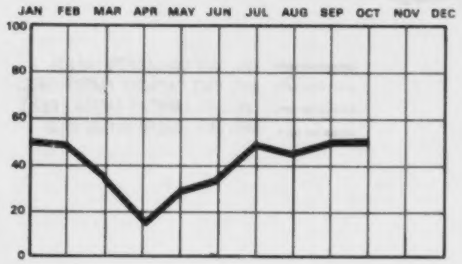
COMBUSTION ENGINEERING



GENERAL ELECTRIC



BABCOCK & WILCOX



NOTE: THIS DISPLAY OF AVERAGE CAPACITY FACTORS PROVIDES A GENERAL PERFORMANCE COMPARISON OF PLANTS SUPPLIED BY THE FOUR NUCLEAR STEAM SUPPLY SYSTEM VENDORS. ONE MUST BE CAREFUL WHEN DRAWING CONCLUSIONS REGARDING THE REASONS FOR THE PERFORMANCE LEVELS INDICATED, SINCE PLANT PERFORMANCE MAY BE AFFECTED BY UNSPECIFIED FACTORS SUCH AS: (1) VARIOUS PLANT DESIGNS AND MODELS ARE INCLUDED FOR EACH VENDOR; (2) TURBINE/GENERATORS AND (3) DIFFERENT ARCHITECT/ENGINEERS ARE ALSO INVOLVED.

GE BWR's	CFMDC
BROWNS FERRY 1	95.5
BROWNS FERRY 2	.0
BROWNS FERRY 3	94.3
BRUNSWICK 1	85.3
BRUNSWICK 2	69.0
COOPER STATION	79.7
DRESDEN 2	45.0
DRESDEN 3	92.6
DUANE ARNOLD	93.9
FITZPATRICK	81.0
HATCH 1	73.4
HATCH 2	80.1
MILLSTONE 1	8.2
MONTICELLO	100.2
NINE MILE POINT 1	95.2
OYSTER CREEK 1	93.4
PEACH BOTTOM 2	102.0
PEACH BOTTOM 3	62.9
PILGRIM 1	74.9
QUAD CITIES 1	.0
QUAD CITIES 2	73.5
VERMONT YANKEE 1	.0

22 UNITS

W PWR's	
BEAVER VALLEY	.0
COOK 1	99.6
COOK 2	54.6
FARLEY 1	98.1
GINNA	100.6
HADDAM NECK	103.5
INDIAN POINT 2	47.3
INDIAN POINT 3	.0
KEWAUNEE	95.9
NORTH ANNA 1	88.4
POINT BEACH 1	75.2
POINT BEACH 2	99.4
PRAIRIE ISLAND 1	9.1
PRAIRIE ISLAND 2	99.1
ROBINSON 2	10.1
SALEM 1	.0
SAN ONOFRE 1	.0
SURRY 1	.0
SURRY 2	98.2
TROJAN	97.6
TURKEY POINT 3	15.1
TURKEY POINT 4	94.8
YANKEE-ROWE 1	1.8
ZION 1	89.8
ZION 2	86.3

25 UNITS

CE PWR's	CFMDC
ARKANSAS 2	92.5
CALVERT CLIFFS 1	45.0
CALVERT CLIFFS 2	85.7
FORT CALHOUN 1	62.0
MAINE YANKEE	98.4
MILLSTONE 2	31.1
PALISADES	104.6
ST LUCIE 1	100.1

8 UNITS

B&W PWR's	CFMDC
ARKANSAS 1	81.1
CRYSTAL RIVER 3	94.7
DAVIS-BESSE 1	.0
OCONEE 1	97.8
OCONEE 2	95.8
OCONEE 3	96.8
RANCHO SECO 1	95.4
THREE MILE ISLAND 1	.0

8 UNITS

	GE BWR's	W PWR's	CE PWR's	B&W PWR's	All PWR's
NET ELECTRICAL PRODUCTION	8,761,933	8,153,794	3,485,299	3,537,699	15,176,792
MDC NET	17,164	18,297	6,069	6,737	31,103
CFMDC	68.5	59.8	77.1	70.5	65.5

Capacity factor in this page, denoted as CFMDC, is a function of the net maximum dependable capacity. See the corresponding definition in the Glossary. The vendor averages are computed by the formula :

$$\frac{\text{Net Electrical Energy Produced by Vendor}}{\text{Potential Electrical Production By Vendor In This Month.}} \times 100\%$$

Units Excluded	Reason for Exclusion
Dresden 1 Humboldt Bay Three Mile Island 2	Unit Shutdown Indefinitely
Big Rock Point	Not Representative of Typical BWRs
LaCrosse	Not Representative of Typical BWRs and Atypical Vendor
Fort St. Vrain	Atypical Vendor and Reactor Design

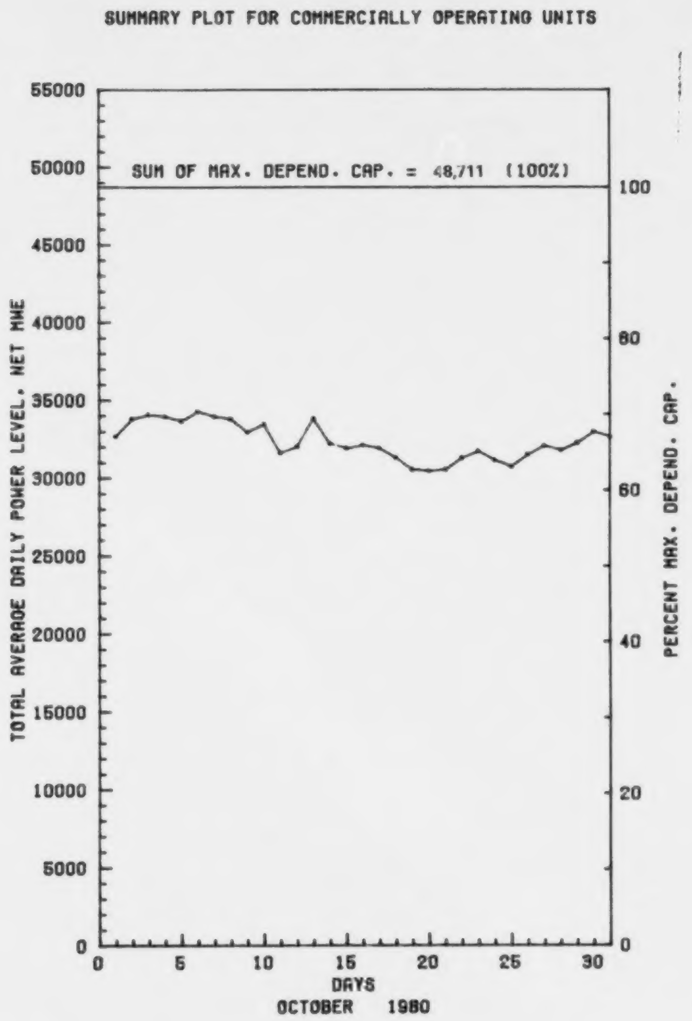
AVERAGE DAILY POWER LEVEL FOR ALL COMMERCIALLY OPERATING UNITS

This chart depicts the average daily power level for the units in commercial operation during the month.

The straight line on the graph labelled "SUM OF MAX. DEPEND. CAP." is plotted at the value shown by summing the separate maximum dependable capacities of the commercially operating units (in Net MWe). The plot shown below the line is calculated by summing the separate average daily power levels of the commercially operating units for each day of the month.

The scale on the left vertical axis runs in 1,000 MWe increments from 0 to 55,000 MWe (Net). The right vertical axis shows the percentage in 10% increments, up to 100% of the "SUM OF MAX. DEPEND. CAP."

It should be recognized that the 100% line would be obtainable only if all of the commercially operating units operated at 100% capacity, 24 hours per day, for the entire month. In other words, since any power generator must occasionally shut down to refuel and/or perform needed maintenance, and also since 100% capacity production is not always required by power demands, the 100% line is a theoretical goal and not a practical one.



ERRATA

Corrections and Amendments to Previously-Reported Data

(NOTE: These changes are reflected in the data contained in the current report)

CORRECTIONS TO NUREG-0020 VOL. 4 NO. 10 OCTOBER 1980

Page	Unit Name	Item	Correction
	NONE		

MULTIPLE - ISSUE CORRECTIONS

MILLSTONE 1

Month	Gross Generation	Net Generation
FEBRUARY 1980	362,000	345,290
JUNE	79,900	73,283
JULY	469,100	446,872
AUGUST 1980	492,600	470,635

OYSTER CREEK

18. NET ELECTRICAL ENERGY (MWH)

JULY 1980	110,500
AUGUST 1980	315,698

SALEM

18. NET ELECTRICAL ENERGY (MWH)

JUNE 1977	19,985
JULY	459,309
AUGUST	628,415
SEPTEMBER	366,934
OCTOBER	-3,414
NOVEMBER	9,190
DECEMBER 1977	572,549
JANUARY 1978	414,471
FEBRUARY 1978	706,321

SECTION 2 – OPERATING POWER REACTORS

ARKANSAS 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1. Location (state/county) (nearest population center)	Arkansas/Pope 6 miles NW of Russellville
2. Electric Reliability Council	Southwest Power Pool
3. Type of Reactor	PWR
4. Power Level (MWT)	2568
5. Design Electrical Rating (MWE Net)	850
6. Date of Initial Criticality	August 6, 1974
7. Date of Commercial Operation	December 19, 1974
8. Condenser Cooling Method	Once Through
9. Condenser Cooling Water Source	Dardenelle Reservoir

Utility & Contractor Information

10. Licensee	Arkansas Power and Light Company
11. Corporate Address	Ninth & Louisiana Streets Little Rock, Arkansas 72203
12. Corporate Contact	W. Cavanaugh III Vice Pres., Generation & Construction
13. Architect/Engineer	Bechtel
14. Nuclear Steam Supply System	B&W
15. Constructor	Bechtel

Regulatory Information

16. IE Region responsible	IV
17. IE Branch Chief	G. L. Madsen
18. IE Section Chief	D. M. Hunnicutt
19. IE Resident Inspector	W. D. Johnson (501) 968-3290
20. Licensing Project Manager	G. S. Vissing
21. Docket Number	50-313
22. License Number/Date of Issue	DPR-51, 5/21/74
23. Public Document Room	Arkansas Polytechnic College Russellville, Arkansas 72801

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

FACILITY ITEMS (PLANS AND PROCEDURES):

MANAGERIAL ITEMS

PLANT STATUS:

POWER OPERATIONS.

LAST IE SITE INSPECTION DATE: SEPTEMBER 22-26, 1980

INSPECTION REPORT NO: 50-313/80-18

SUMMARY

INSPECTION SUMMARY

INSPECTION CONDUCTED DURING PERIOD OF JUNE 17-20, 1980 AND JULY 14-18, 1980 (80-12): ROUTINE, UNANNOUNCED INSPECTION OF LICENSEE ACTION ON PREVIOUS INSPECTION FINDINGS; LICENSEE INTERNAL AUDITS; QUALITY ASSURANCE PROGRAM FOR PROCUREMENT, RECEIPT, HANDLING AND STORAGE OF MATERIAL; MAINTENANCE RECORDS; LICENSED OPERATOR REQUALIFICATION PROGRAM; FIRE BRIGADE TRAINING; GENERAL TRAINING; AND TRAINING RECORDS. WITHIN THE EIGHT (8) AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE FOUND IN FOUR (4) AREAS; FOUR (4) APPARENT ITEMS OF NONCOMPLIANCE WERE FOUND IN FOUR (4) AREAS (INFRACTION - FAILURE TO TAKE PROMPT CORRECTIVE ACTION; INFRACTION - REQUALIFICATION TRAINING PROGRAM REQUIREMENTS NOT MET; INFRACTION - FIRE BRIGADE TRAINING NOT COMPLETE; DEFICIENCY - FAILURE TO MAINTAIN RECORDS.

INSPECTION CONDUCTED DURING PERIOD SEPTEMBER 2-5, 1980 (80-16): ROUTINE, UNANNOUNCED INSPECTION INCLUDING AUDIT PROGRAMS, RECORDS, AND DOCUMENT CONTROL. WITHIN THE THREE AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE WERE IDENTIFIED IN TWO AREAS; ONE APPARENT ITEM OF NONCOMPLIANCE (INFRACTION - FAILURE TO FOLLOW PROCEDURES WITH REGARD TO AUDIT FINDINGS WAS IDENTIFIED IN ONE AREA.

INSPECTION CONDUCTED DURING PERIOD OF SEPTEMBER 22-26, 1980 (80-18): ROUTINE, UNANNOUNCED INSPECTION INCLUDING FOLLOW-UP ON IE BULLETINS. WITHIN THE AREA INSPECTED, NO ITEMS OF NONCOMPLIANCE AND ONE DEVIATION WERE IDENTIFIED (DEVIATION - SAMPLING PROGRAM).

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-31 03L-0	8/27/80	9/25/80	DIESEL GENERATOR EXHAUST FLANGE LEAK
80-32 04L-0	9/5/80	10/1/80	GASEOUS EFFLUENT RELEASE RATE
80-33 03L-0	9/6/80	10/2/80	RCS MAKEUP BLOCK VALVE FAILURE
80-34 03L-0	9/5/80	10/1/80	STEAM GENERATOR TUBE LEAK
80-35 99X-0	9/6/80	10/24/80	REACTOR BUILDING COOLING COILS LOW FLOW
80-36 03L-0	9/29/80	10/23/80	FAILURE TO PERFORM NUCLEAR INSTRUMENTATION CALIBRATION
80-37 03L-0	9/29/80	10/23/80	LEAD HYDROGEN PURGE SYSTEM FAILURE

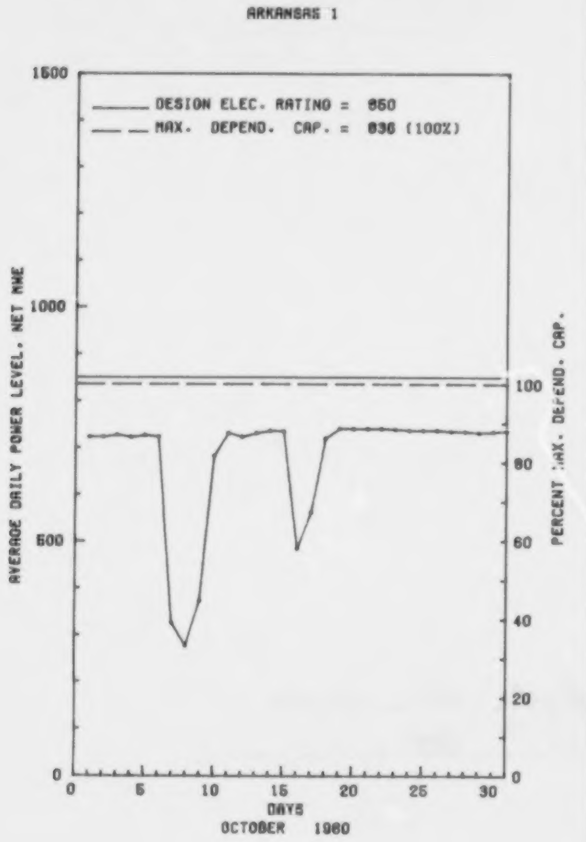
REVIEWED BY OMPA LOEB: RE Bobe **DATE:** 12/10/80

REVIEWED BY NRR DL: **DATE:** 12/10/80

GPO 873-667

ARKANSAS 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-313
2. Utility Contact: L. S. Bramlett (501) 968-2519
3. Licensed Thermal Power (MWt): 2568
4. Nameplate Rating (Gross MWe): 1003 x 0.9 = 903
5. Design Electrical Rating (Net MWe): 850
6. Maximum Dependable Capacity (Gross MWe): 883
7. Maximum Dependable Capacity (Net MWe): 836
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	745.0	7,320.0	51,451.0
12. Hours Reactor Was Critical	742.0	4,199.2	34,337.4
13. Reactor Reserve Shutdown Hours	.0	647.6	4,895.0
14. Hours Generator On-Line	738.9	4,130.7	33,601.2
15. Unit Reserve Shutdown Hours	.0	20.8	817.5
16. Gross Thermal Energy (MWH)	1,734,256	9,825,000	81,087,066
17. Gross Electrical Energy (MWH)	531,799	3,025,419	26,746,575
18. Net Electrical Energy (MWH)	504,847	2,876,814	25,510,850
19. Unit Service Factor	99.2	56.4	65.3
20. Unit Availability Factor	99.2	56.7	66.9
21. Unit Capacity Factor (MDC Net)	81.1	47.0	59.3
22. Unit Capacity Factor (DER Net)	79.7	46.2	58.3
23. Unit Forced Outage Rate	.0	31.1	17.9

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
JANUARY 2, 1981 TO MARCH 15, 1981 - REFUELING.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-10	10/07/80	F	6.1	G	3		ZZ	ZZZZZZ	THE "A" MAIN FEED PUMP WAS ACCIDENTALLY TRIPPED WHEN A MECHANIC, WHO WAS CLEANING UP OIL AROUND THE PUMP, SLIPPED AND HIT THE TRIP MECHANISM.
80-11	10/07/80	F	.0	B	5		CH	MECFUN	POWER WAS LIMITED BECAUSE OF MAINTENANCE ON THE "A" MAIN FEED PUMP GOVERNOR CONTROL MECHANISM. THE CAUSE WAS DETERMINED TO BE TRASH IN THE CONTROL OIL SYSTEM.
80-12	10/16/80	F	.0	B	5		CH	MECFUN	POWER WAS LIMITED BECAUSE OF MAINTENANCE ON THE "A" MAIN FEEDPUMP GOVERNOR CONTROL MECHANISM. THE CAUSE WAS DETERMINED TO BE TRASH IN THE CONTROL OIL SYSTEM.

SUMMARY

ARKANSAS 1 OPERATED WITH ONE OUTAGE FOR OPERATIONAL ERROR AND TWO REDUCTIONS FOR MAINTENANCE DURING OCTOBER. REFUELING IS ANTICIPATED TO BEGIN ON JANUARY 2, 1981.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5 Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

ARKANSAS 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Arkansas/Pope 6 miles NW of Russellville
2.	Electric Reliability Council	Southwest Power Pool
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2815
5.	Design Electrical Rating (MWE Net)	912
6.	Date of Initial Criticality	12-5-78
7.	Date of Commercial Operation	3/26/80
8.	Condenser Cooling Method	Recirculating
9.	Condenser Cooling Water Source	Cooling Tower

Utility & Contractor Information

10.	Licensee	Arkansas Power & Light Company
11.	Corporate Address	Ninth & Louisiana Streets Little Rock, Arkansas 72203
12.	Corporate Contact	W. Cavanaugh III Vice Pres., Generation & Construction
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Combustion Engineering, Inc.
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	IV
17.	IE Branch Chief	G. L. Madsen
18.	IE Section Chief	D. M. Hunnicutt
19.	IE Resident Inspector	W. D. Johnson (501) 968-3290
20.	Licensing Project Manager	R. E. Martin
21.	Docket Number	50-368
22.	License Number/Date of Issue	NFP-6, 9/1/78
23.	Public Document Room	Arkansas Polytechnic College Russellville, Arkansas 72801

INSPECTION STATUS

ENFORCEMENT STATUS

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

FACILITY ITEMS (PLANS AND PROCEDURES):

MANAGERIAL ITEMS:

PLANT STATUS:

THE UNIT RETURNED TO POWER OPERATIONS FOLLOWING AN OUTAGE FOR REMOVAL OF ASIAN CLAMS, SILT AND CORROSION PRODUCTS FROM THE SERVICE WATER SYSTEM.

LAST IE INSPECTION DATE: SEPTEMBER 22-26, 1980 +

INSPECTION REPORT NO: 50-368/80-18 +

SUMMARY

INSPECTION SUMMARY

+ INSPECTION CONDUCTED DURING PERIOD OF JUNE 17-20, 1980 AND JULY 14-18, 1986 (80-12): ROUTINE, UNANNOUNCED INSPECTION OF LICENSEE ACTION ON PREVIOUS INSPECTION FINDINGS; LICENSEE INTERNAL AUDITS; QUALITY ASSURANCE PROGRAM FOR PROCUREMENT, RECEIPT, HANDLING AND STORAGE OF MATERIAL; MAINTENANCE RECORDS; LICENSED OPERATOR REQUALIFICATION PROGRAM; FIRE BRIGADE TRAINING; GENERAL TRAINING; AND TRAINING RECORDS. WITHIN THE EIGHT (8) AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE FOUND IN FOUR (4) AREAS; FOUR (4) APPARENT ITEMS OF NONCOMPLIANCE WERE FOUND IN FOUR (4) AREAS (INFRACTION - FAILURE TO TAKE PROMPT CORRECTIVE ACTION; INFRACTION - REQUALIFICATION TRAINING PROGRAM REQUIREMENTS NOT MET; INFRACTION - FIRE BRIGADE TRAINING NOT COMPLETE; DEFICIENCY - FAILURE TO MAINTAIN RECORDS.

INSPECTION CONDUCTED DURING PERIOD SEPTEMBER 2-5, 1980 (80-16): ROUTINE, UNANNOUNCED INSPECTION INCLUDING AUDIT PROGRAM, RECORDS, AND DOCUMENT CONTROL. WITHIN THE THREE AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE WERE IDENTIFIED IN TWO AREAS; ONE APPARENT ITEM OF NON COMPLIANCE (INFRACTION - FAILURE TO FOLLOW PROCEDURES WITH REGARD TO AUDIT FINDINGS WAS IDENTIFIED IN ONE AREA.

INSPECTION CONDUCTED DURING PERIOD OF SEPTEMBER 22-26, 1980 (80-18): ROUTINE, UNANNOUNCED INSPECTION INCLUDING SURVEILLANCE OF CORE POWER DISTRIBUTION LIMITS, CORE THERMAL POWER EVALUATION, FOLLOW-UP ON IE BULLETINS. WITHIN THE THREE (3) AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE AND ONE DEVIATION WERE IDENTIFIED (DEVIATION - SAMPLING PROGRAM).

REPORTS RECEIVED FROM LICENSEE

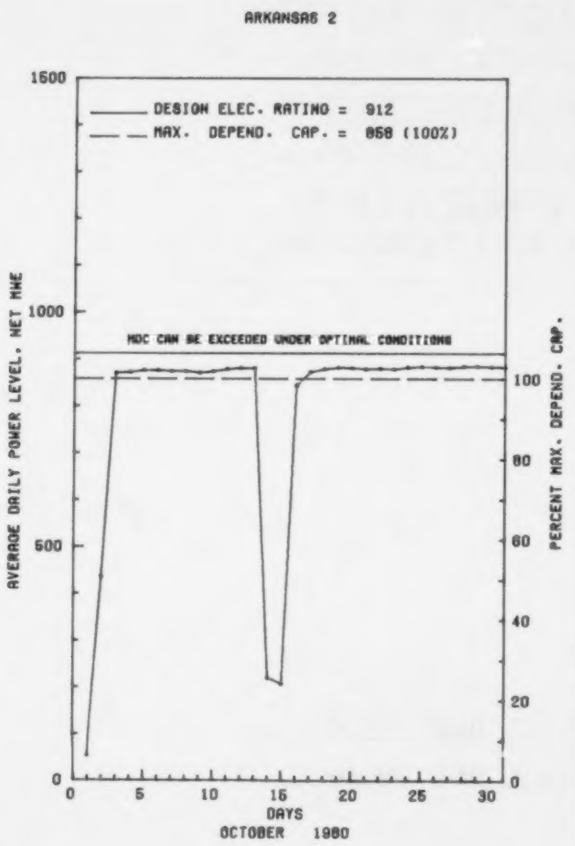
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-69 03L-0	9/3/80	10/3/80	SODIUM HYDROXIDE ADDITION TANK CONCENTRATION

REVIEWED BY OMPA LOEB: MR Beebe DATE: 12/10/80

REVIEWED BY NRR DL: R E Martin DATE: 12/11/80

ARKANSAS 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-368
2. Utility Contact: Rex Pendergraft (501) 968-2519
3. Licensed Thermal Power (MWt): 2815
4. Nameplate Rating (Gross MWe): 959
5. Design Electrical Rating (Net MWe): 912
6. Maximum Dependable Capacity (Gross MWe): 897
7. Maximum Dependable Capacity (Net MWe): 858
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	745.0	5,280.0	5,280.0
12. Hours Reactor Was Critical	703.6	3,942.1	3,942.1
13. Reactor Reserve Shutdown Hours	26.6	701.1	701.1
14. Hours Generator On-Line	700.3	3,877.0	3,877.0
15. Unit Reserve Shutdown Hours	.0	75.0	75.0
16. Gross Thermal Energy (MWH)	1,899,722	9,877,702	9,877,702
17. Gross Electrical Energy (MWH)	618,173	3,207,523	3,207,523
18. Net Electrical Energy (MWH)	591,465	3,058,832	3,058,832
19. Unit Service Factor	94.0	73.4	73.4
20. Unit Availability Factor	94.0	74.9	74.9
21. Unit Capacity Factor (MDC Net)	92.5	67.5	67.5
22. Unit Capacity Factor (DER Net)	87.1	63.5	63.5
23. Unit Forced Outage Rate	6.0	26.6	26.6

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): MARCH 15, 1981 TO MAY 15, 1981 - REFUELING.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-22	10/01/80	F	16.4	A	9		CB	PUMPXX	LOST "B" RCP WHILE AT APPROXIMATELY 27% POWER. PUMP TRIP WAS PROBABLY DUE TO MAINTENANCE IN THE PUMP BREAKER CABINET.
80-23	10/14/80	F	28.3	A	3		CH	INSTRU	SQUARE ROOT EXTRACTOR IN THE "A" LOOP FW CONTROL SYSTEM FAILED CAUSING AN OVERFEED SITUATION. THE RX TRIPPED ON HIGH "A" S/G LEVEL. INSTRUMENT WAS REPLACED.

SUMMARY

ARKANSAS 2 OPERATED NORMALLY IN OCTOBER, WITH TWO OUTAGES ON THE 1ST AND 14TH DUE TO EQUIPMENT FAILURE.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

NRC POWER RESTRICTIONS: NONE

BEAVER VALLEY 1

FACILITY DATA

Facility Description		
1.	Location (state/countv) (nearest population center)	Pennsylvania/Beaver 5 miles E of E. Liverpool, Ohio
2.	Electric Reliability Council	East Central Area Reliability Coordination Agreement
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2660
5.	Design Electrical Rating (MWE Net)	852
6.	Date of Initial Criticality	May 10, 1976
7.	Date of Commercial Operation	10/01/76 (50%) 04/30/77 (100%)
8.	Condenser Cooling Method	Cooling tower/Closed loop
9.	Condenser Cooling Water Source	Ohio River
Utility & Contractor Information		
10.	Licensee	Duquesne Light Company
11.	Corporate Address	435 Sixth Avenue Pittsburgh, Pennsylvania 15219
12.	Corporate Contact	C. N. Dunn, Vice President
13.	Architect/Engineer	Stone & Webster
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Stone & Webster
Regulatory Information		
16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Section Chief	E. C. MCCABE
19.	IE Resident Inspector	D. A. BECKMAN (412-643-9200/9201)
20.	Licensing Project Manager	D. A. Chaney
21.	Docket Number	50-334
22.	License Number/Date of Issue	DPR-66, 1/30/76
23.	Public Document Room	B. F. Jones Memorial Library 633 Franklin Avenue Alaquippe, PA 15001

INSPECTION STATUS

TECHNICAL SPECIFICATION 3.11.3.4 REQUIRES IN PART THAT A CONTINUOUS FIRE WATCH SHALL BE ESTABLISHED IMMEDIATELY IF C02 FIRE PROTECTION IS LOST TO A CABLE SPREADING ROOM. CONTRARY TO THE ABOVE, ON AUGUST 5, 1980 FROM APPROXIMATELY 11:00 A.M. TO 11:45 A.M. A CONTINUOUS FIRE WATCH WAS NOT PROVIDED FOR THE CABLE SPREADING ROOMS WHILE THE C02 SYSTEM WAS INOPERABLE FOR REPAIRS.
(8034 INFRACTION)

TENNESSEE VALLEY AUTHORITY'S RESPONSE TO IE BULLETIN 80-17, SUPPLEMENT 1, DATED AUGUST 18, 1980 FOR BROWNS FERRY UNIT 1, 2 AND 3, STATES IN PART, THAT THE UT SENSORS ARE CONTINUOUSLY CONNECTED TO A RECORDER WHICH MONITOR FOR AN ACCUMULATION OF WATER IN THE EAST AND WEST SCRAM DISCHARGE HEADERS. THESE RECORDERS ARE CHECKED AT 30 MINUTE INTERVALS. CONTRARY TO THE ABOVE, ON AUGUST 21, 1980, THE RECORDERS WERE NOT CHECKED FROM 11:30 A.M. TO 12:45 P.M. FOR A TOTAL OF ONE HOUR AND 15 MINUTES.
(8034 DEVIATION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

- TWO OF THE THREE PRESSURIZER POWER OPERATED RELIEF VALVES (PORVS) WERE FOUND LEAKING DURING CURRENT PLANT HEATUP AND WERE ISOLATED BY THEIR RESPECTIVE BLOCK VALVES.

FACILITY ITEMS (PLANS AND PROCEDURES):

- AFTER A REFUELING AND MAJOR MODIFICATION OUTAGE, PLANT HEATUP BEGAN ON OCTOBER 22.

PLANT STATUS:

- IN HOT STANDBY, WORKING ON POST-OUTAGE MATERIAL PROBLEMS.

- LAST IE SITE INSPECTION DATE: 10/29-31/80

INSPECTION REPORT NO: 50-334/80-27

SUMMARY

INSPECTION SUMMARY

- 50-334/80-24 - AUG 25-29: ROUTINE INSPECTION BY ONE REGION-BASED INSPECTOR (32 HRS) OF DESIGN CHANGE APPROVAL, REVIEW, IMPLEMENTATION, AND MANAGEMENT CONTROLS; QUALITY ASSURANCE AND QUALITY CONTROL INTERFACE WITH DESIGN CHANGES; AND ONSITE SAFETY COMMITTEE MEETING MINUTES. NO NONCOMPLIANCES WERE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

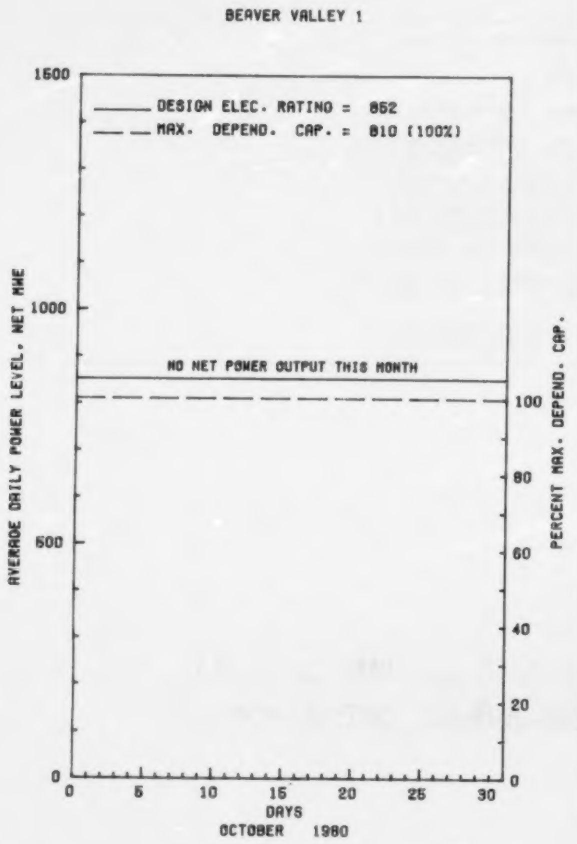
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-004/ SP	09/25/80	09/26/80	PORTION OF FIRE MAIN DUG UP BY BACKHOE. SECTION ISOLATED AND FIRE WATCHES POSTED. TEMPORARY CROSS CONNECT MADE TO UNAFFECTED PORTION OF SYSTEM. DAMAGED SECTION REPLACED.
80-024/ 03L	04/28/80	09/30/80	ORC AUDIT IDENTIFIED MISSED SURVEILLANCE TEST (WEEKLY) ON BORIC ACID TRANSFER PUMPS DUE TO SCHEDULING ERROR. SCHEDULING MODIFICATIONS MADE.
80-061/ 03L	08/29/80	09/26/80	BORATION FLOW PATH INOPERABLE IN MODE 5 DUE TO OPERATOR ERROR. VALVE LINEUPS PERFORMED WITHOUT USE OF PROCEDURE. SPECIAL OPER ORDER ISSUED TO ENSURE USE OF PROCEDURES FOR SR SYSTEMS.
80-069/ 01T	09/20/80	10/03/80	MOTOR AND DIESEL DRIVEN FIRE PUMPS INOPERABLE SIMULTANEOUSLY MOTOR DRIVEN PUMP MOTOR FAILED DUE TO LOSS OF COOLING WATER DIESEL PUMP OOS FOR MODIFICATION WAS IMMED RETURN TO SERVICE.
80-071/ 01T	09/26/80	10/13/80	CONTROL ROOM ISOLATION TIME EXCEED TECH SPEC LIMIT. TS FAILED TO INCLUDE DIESEL GENERATOR SEQUENCING TIME. REQUEST FOR TECH SPEC AMENDMENT TO BE SUBMITTED. CHANGE FROM 30 TO 77 SEC.
80-073/ 01T	10/03/80	10/17/80	POTENTIAL FOR MULTIPLE STEAM GENERATOR BLOWDOWN FROM SINGLE LINE BREAK ON RESIDUAL HEAT RELEASE HEADER OR AUX FEED PUMP SUPPLY HEADER. ENGINEERING STUDY INITIATED.

REVIEWED BY OMPA LOEB: R Bobe **DATE:** 12/10/80

REVIEWED BY NRR DL: DAChaney **DATE:** 12/10/80

BEAVER VALLEY 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-334
2. Utility Contact: DAVID R. TIMKO (412) 643-5308
3. Licensed Thermal Power (MWt): 2660
4. Nameplate Rating (Gross MWe): 1026 x 0.9 = 923
5. Design Electrical Rating (Net MWe): 852
6. Maximum Dependable Capacity (Gross MWe): 845
7. Maximum Dependable Capacity (Net MWe): 810
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	745.0	7,320.0	39,480.0
12. Hours Reactor Was Critical	.0	.0	13,774.7
13. Reactor Reserve Shutdown Hours	.0	.0	4,482.7
14. Hours Generator On-Line	.0	.0	13,105.0
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	0	26,974,253
17. Gross Electrical Energy (MWH)	0	0	8,277,940
18. Net Electrical Energy (MWH)	0	0	7,570,743
19. Unit Service Factor	.0	.0	33.9
20. Unit Availability Factor	.0	.0	33.9
21. Unit Capacity Factor (MDC Net)	.0	.0	26.5
22. Unit Capacity Factor (DER Net)	.0	.0	25.2
23. Unit Forced Outage Rate	.0	.0	46.4

24. Shutdowns Scheduled Over Next 6 Months (Type,Date,and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 11/07/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
1	01/01/80	S	745.0	D	4		ZZ	ZZZZZZ	UNIT SHUTDOWN FOR MAJOR MODIFICATIONS AS REQUIRED BY THE NUCLEAR REGULATORY COMMISSION, INCLUDING NRC BULLETINS IEB 79-02 AND 79-14.

SUMMARY

BEAVER VALLEY 1 REMAINED SHUTDOWN FOR REQUIRED NRC SYSTEM MODIFICATIONS PER BULLETINS 79-02 AND 79-14.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

BIG ROCK POINT 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Michigan/Charlevoix 4 miles NE of Charlevoix
2.	Electric Reliability Council	East Central Area Reliability Coordination Agreement
3.	Type of Reactor	BWR
4.	Power Level (MWT)	240
5.	Design Electrical Rating (MWE Net)	72
6.	Date of Initial Criticality	September 27, 1962
7.	Date of Commercial Operation	March 29, 1963
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10.	Licensee	Consumers Power Company
11.	Corporate Address	212 West Michigan Avenue Jackson, Michigan 49201
12.	Corporate Contact	R. B. DeWitt
13.	Architect/Engineer	Bechtel Corporation
14.	Nuclear Steam Supply System	GE
15.	Constructor	Bechtel Corporation

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	D. C. BOYD
19.	IE Resident Inspector	G. Wright
20.	Licensing Project Manager	W. A. Paulson
21.	Docket Number	50-155
22.	License Number/Date of Issue	DPR-6 issued 8/30/62
23.	Public Document Room	Charlevoix Public Library 107 Clinton Street Charlevoix, Michigan

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS.

- UNIT SHUTDOWN ON OCTOBER 31, 1980, FOR A FIVE WEEK REFUELING OUTAGE.
- LAST IE SITE INSPECTION DATE: SEPTEMBER 22-26, 1980
- INSPECTION REPORT NO: 80-12

SUMMARY

INSPECTION SUMMARY

- INSPECTION ON SEPTEMBER 17-19, (80-11): MATERIAL CONTROL AND ACCOUNTABILITY INCLUDING FACILITY ORGANIZATION AND OPERATIONS, MEASUREMENT AND CONTROL, STORAGE AND INTERNAL CONTROLS, SHIPPING AND RECEIVING, RECORDS AND REPORTS, INVENTORY AND INVENTORY VERIFICATION AND MANAGEMENT OF THE MATERIALS CONTROL SYSTEM. THE INSPECTION INVOLVED 40 INSPECTOR-HOURS ONSITE BY 2 NRC INSPECTORS, AND WAS BEGUN DURING THE REGULAR HOURS. THE LICENSEE WAS FOUND TO BE IN COMPLIANCE WITH NRC REQUIREMENTS IN THE SEVEN AREAS EXAMINED DURING THE INSPECTION.
- INSPECTION ON SEPTEMBER 22-26, (80-12): INCLUDED A REVIEW OF SECURITY PLAN; SECURITY PROGRAM AUDIT; SECURITY ORGANIZATION; PROTECTED AREA PHYSICAL BARRIERS; VITAL AREA PHYSICAL BARRIERS; LIGHTING; ACCESS CONTROL (IDENTIFICATION, AUTHORIZATION, AND BADGING); ACCESS CONTROL (SEARCH); ACCESS CONTROL (ESCORTING); ACCESS CONTROL (VITAL AREAS); ALARM STATIONS; DETECTION AIDS; ASSESSMENT AIDS; COMMUNICATIONS; TESTING AND MAINTENANCE; COMPENSATORY MEASURES; POWER SUPPLY; RESPONSE; LOCKS, KEYS, AND COMBINATIONS; RECORDS AND REPORTS; FOLLOWUP ON PREVIOUS ITEMS OF NONCOMPLIANCE/DEVIATIONS AND INDEPENDENT INSPECTION EFFORTS. THE INSPECTION INVOLVED 66.5 INSPECTOR-HOURS ONSITE BY 2 NRC INSPECTORS. THE INSPECTION BEGAN DURING THE DAY SHIFT PERIOD; 16 INSPECTOR-HOURS WERE ACCOMPLISHED DURING OFFSHIFT PERIODS. THE LICENSEE WAS FOUND TO BE IN COMPLIANCE WITH NRC REQUIREMENTS WITHIN THE AREAS EXAMINED DURING THIS INSPECTION EXCEPT FOR THE FOLLOWING ITEM: ACCESS CONTROL - (IDENTIFICIATION, AUTHORIZATION, AND BADGING) - THE LICENSEE DID NOT CONDUCT PROPER BACKGROUND SCREENING OF CONTRACTOR PERSONNEL. THE ITEMS OF NONCOMPLIANCE FROM THE PREVIOUS SECURITY INSPECTION (IE INSPECTION REPORT NO. 50-155/80-05), WITH THE EXCEPTION OF ONE ITEM, WERE CLOSED. IN ADDITION, SEVERAL UNRESOLVED ITEMS WERE IDENTIFIED WHICH WILL BE REFERRED TO NRC HEADQUARTERS FOR POSSIBLE RESOLUTION.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

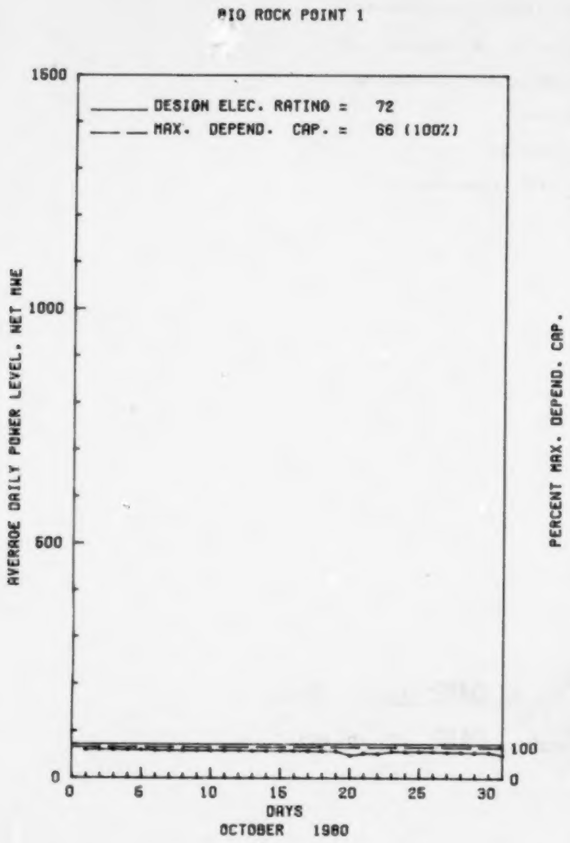
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-25/ 03L-0	08/19/80	09/12/80	RDS INSTRUMENT CHANNEL D OUT OF SERVICE LT-3183 VENTING.
80-26/ 03L-0	08/20/80	09/12/80	RDS INSTRUMENT CHANNEL D OUT OF SERVICE LT-3183 VENTING.
80-27/ 03L-0	08/29/80	09/12/80	RDS INSTRUMENT CHANNEL D OUT OF SERVICE LT-3183 VENTING.
80-28/ 03L-0	09/09/80	10/07/80	CHANNEL D OF REACTOR DEPRESS. SYSTEM RDS LOOPS WERE OPERABLE WITH 2/3 LOGIC INSTEAD OF 2/4.
80-29/ 03L-0	09/11/80	10/07/80	CELL 12 OF REACTOR DEPRESSURIZING BATTERY FOR CHANNEL D HAD A LOW READING.
80-30/ 03L-0	09/19/80	10/17/80	MINOR THROUGH SEAT LEAKAGE IN EXCESS OF T.S. WAS OBSERVED IN ISOLATION VALVE CV4027.
80-31/ 03L-0	09/23/80	10/20/80	REACTOR LEVEL TRANSMITTER OUT OF CALIBRATION.
80-32/ 03L-0	09/17/80	10/15/80	ALL FOUR RDS LOOPS REMAINED OPERABLE WITH 2/3 LOGIC INSTEAD OF 2/4 DURING TEST, OF SENOR CHANNEL D.
80-33/ 01T-0	10/03/80	10/15/80	A MINOR LEAK OCCURRED AT THE CLEAN-UP PUMP IN THE REACTOR CLEAN-UP DEMINERALIZER SYSTEM.

REVIEWED BY OMPA LOEB: John J. Ruzzo **DATE:** 12-10-80

REVIEWED BY NRR DL: Walter A. Paulson **DATE:** 12-10-80

BIG ROCK POINT 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-155
2. Utility Contact: G. Himebauch (616) 547-6537 x180
3. Licensed Thermal Power (MWt): 240
4. Nameplate Rating (Gross MWe): 70.6 x 0.85 = 60
5. Design Electrical Rating (Net MWe): 72
6. Maximum Dependable Capacity (Gross MWe): 70
7. Maximum Dependable Capacity (Net MWe): 66
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

ITEM 21 CALCULATED WITH A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	745.0	7,320.0	154,243.0
12. Hours Reactor Was Critical	744.7	6,960.1	107,093.1
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	744.4	6,932.9	104,931.8
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	147,106	1,403,268	19,521,199
17. Gross Electrical Energy (MWH)	45,333	429,172	6,165,090
18. Net Electrical Energy (MWH)	42,794	405,450	5,833,787
19. Unit Service Factor	99.9	94.7	68.0
20. Unit Availability Facter	99.9	94.7	68.0
21. Unit Capacity Factor (MDC Net)	87.0	83.9	55.9
22. Unit Capacity Factor (DER Net)	79.8	76.9	52.5
23. Unit Forced Outage Rate	.0	.6	22.3

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 12/24/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type^1	Duration (Hours)	Reason^2	Method of Shutting Down Reactor^3	Licensee Event Report No.	System Code^4	Component Code^5	Cause & Corrective Action to Prevent Recurrence
10	10/31/80	S	.6	C	1				ORDERLY SHUTDOWN FOR REFUELING.

SUMMARY

BIG ROCK POINT 1 OPERATED NORMALLY UNTIL THE 31ST, WHEN THE PLANT ENTERED A REFUELING OUTAGE.

LEGEND

^{1}Type:
F–Forced
S–Scheduled

^{2}Reason:
A–Equipment Failure
B–Maintenance or Test
C–Refueling
D–Regulatory Restriction
E–Operator Training & License Examination
F–Administrative
G–Operational Error
H–Other

^{3}Method:
1–Manual
2–Manual Scram
3–Automatic Scram
4–Continuations
5–Load Reduction
9–Other

^{4}Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

^{5}Exhibit H: Same source.

BROWNS FERRY 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Alabama/Limestone 10 miles NW of Decatur
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	3293
5.	Design Electrical Rating (MWE Net)	1065
6.	Date of Initial Criticality	August 17, 1973
7.	Date of Commercial Operation	August 1, 1974
8.	Condenser Cooling Method	Once Through With Forced Draft Cooling Towers
9.	Condenser Cooling Water Source	Tennessee River

Utility & Contractor Information

10.	Licensee	Tennessee Valley Authority
11.	Corporate Address	500A Chestnut Street Tower II Chattanooga, Tennessee 37401
12.	Corporate Contact	H. G. Parris, Manager of Power.
13.	Architect/Engineer	TVA
14.	Nuclear Steam Supply System	GE
15.	Constructor	TVA

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. Lewis
18.	IE Section Chief	H. C. Dance
19.	IE Resident Inspector	R. Sullivan (205) 729-6196
20.	Licensing Project Manager	R. Clark
21.	Docket Number	50-259
22.	License Number/Date of Issue	DPR-33, 6/26/73
23.	Public Document Room	Athens Public Library South and Forrest Athens, Alabama 35611

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

100 PERCENT FULL POWER OPERATION.

+ LAST IE SITE INSPECTION DATE: AUGUST 1-31, 1980

+ INSPECTION REPORT NO: 50-259/80-34

SUMMARY

INSPECTION SUMMARY

+ INSPECTION AUGUST 1-31 (80-34): THIS ROUTINE INSPECTION INVOLVED 43 RESIDENT INSPECTOR-HOURS IN THE AREAS OF OPERATIONAL SAFETY, IE BULLETIN FOLLOWUP, IE CIRCULAR FOLLOWUP, REPORTABLE OCCURRENCES, REACTOR TRIP FOLLOWUP, PLANT PHYSICAL PROTECTION, MAINTENANCE, FIRE PROTECTION, CORE PHYSICS AND ORGANIZATION. OF THE 10 AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE FOUND IN 8 AREAS; ONE APPARENT ITEM OF NONCOMPLIANCE AND ONE APPARENT DEVIATION WAS FOUND IN 2 AREAS (INFRACTION - FAILURE TO PROVIDE A CONTINUOUS FIRE WATCH IN THE CABLE SPREADING ROOMS WITH THE CARBON DIOXIDE SYSTEM INOPERABLE; DEVIATION - FAILURE TO MONITOR SCRAM DISCHARGE HEADER LEVEL EVERY 30 MINUTES AS COMMITTED IN RESPONSE TO IEB 80-17 SUPPLEMENT 1).

REPORTS RECEIVED FROM LICENSEE

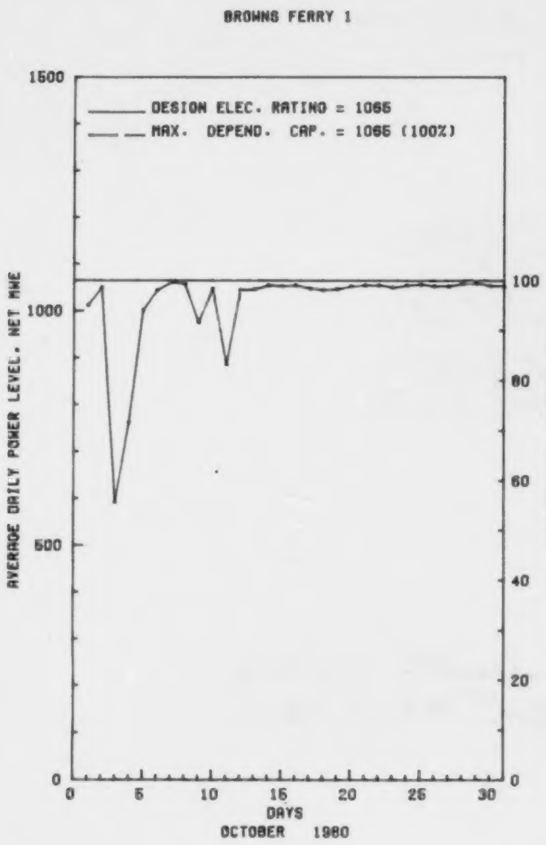
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-060/ 03L-0	08/14/80	09/12/80	THERMOSTATIC SETTING FOR CORE SPRAY PUMP AREA COOLER FAN ON HIGH.
80-061/ 03L-0	08/14/80	09/12/80	BREAKER TRIPPED FOR 1A RESIDUAL HEAT REMOVAL AREA COOLER FAN.
80-063/ 03L-0	08/20/80	09/16/80	TURBINE VENT REMOVED FROM SERVICE EXCEED TIME REQUIREMENTS.
80-064/ 03L-0	08/20/80	09/17/80	TURBINE BUILDING ROOT FAN OUT OF SERVICE LONGER THAN ALLOWED BY TECHNICAL SPECIFICATION.
80-066/ 03L-0	09/02/80	09/29/80	FAILED RELAY HYDROGEN MONITOR FOR TORUS.
80-068/ 01T-0	09/01/80	09/22/80	FIRE PROTECTION SYSTEM AUDIT NOT CONDUCTED.
80-069/ 03L-0	09/02/80	09/26/80	REFUELING ZONE MODULATING DAMPER FAILED TO CLOSE DURING TEST.
80-070/ 03L-0	09/19/80	10/01/80	FUEL ASSEMBLY MISORIENTATION.

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 12-10-80

REVIEWED BY NRR DL: Dick Clark **DATE:** 12-10-80

BROWNS FERRY 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-259
2. Utility Contact: Don Green (205) 729-6846
3. Licensed Thermal Power (MWt): 3293
4. Nameplate Rating (Gross MWe): 1280 x 0.9 = 1152
5. Design Electrical Rating (Net MWe): 1065
6. Maximum Dependable Capacity (Gross MWe): 1098
7. Maximum Dependable Capacity (Net MWe): 1065
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	745.0	7,320.0	54,818.0
12. Hours Reactor Was Critical	738.1	5,094.4	33,371.7
13. Reactor Reserve Shutdown Hours	6.9	380.0	5,069.5
14. Hours Generator On-Line	735.5	4,957.0	32,569.9
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	2,321,167	14,751,985	88,888,478
17. Gross Electrical Energy (MWH)	778,800	4,775,900	29,294,360
18. Net Electrical Energy (MWH)	757,925	4,630,063	28,432,031
19. Unit Service Factor	98.7	67.7	59.4
20. Unit Availability Factor	98.7	67.7	59.4
21. Unit Capacity Factor (MDC Net)	95.5	59.4	48.7
22. Unit Capacity Factor (DER Net)	95.5	59.4	48.7
23. Unit Forced Outage Rate	1.3	8.7	30.2

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
REFUEL OUTAGE APRIL, 1981.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
148	10/03/80	F	9.5	G	3				TURBINE TRIP ON LOW OIL LEVEL DUE TO PERSONNEL INADVERTENTLY CHECKING LEVEL SWITCH.
149	10/09/80	F	.0	B	5				DERATED FOR MAINTENANCE TO "A" REACTOR FEEDWATER PUMP.
150	10/10/80	F	.0	B	5				DERATED FOR MAINTENANCE TO "A" REACTOR FEEDWATER PUMP AND TURBINE C.V. TESTS AND SI'S.
151	10/15/80	S	.0	H	5				DERATED FOR CONTROL ROD PATTERN ADJUSTMENT.
152	10/17/80	S	.0	B	5				DERATED FOR TURBINE C.V. TESTS AND SI'S.
153	10/25/80	S	.0	B	5				DERATED FOR TURBINE C.V. TESTS AND SI'S.

SUMMARY

BROWNS FERRY 1 OPERATED AT NEAR FULL POWER FOR THE MAJORITY OF OCTOBER WITH ONE OUTAGE AND FIVE REDUCTIONS LISTED IN DETAIL ABOVE

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

BROWNS FERRY 2

FACILITY DATA

Facility Description

1. Location (state/county) (nearest population center): Alabama/Limestone 10 miles NW of Decatur
2. Electric Reliability Council: Southeastern Electric Reliability Council
3. Type of Reactor: BWR
4. Power Level (MWT): 3293
5. Design Electrical Rating (MWE Net): 1065
6. Date of Initial Criticality: July 20, 1974
7. Date of Commercial Operation: March 1, 1975
8. Condenser Cooling Method: Once Through With Forced Draft Cooling Towers
9. Condenser Cooling Water Source: Tennessee River

Utility & Contractor Information

10. Licensee: Tennessee Valley Authority
11. Corporate Address: 500A Chestnut Street Tower II Chattanooga, Tennessee 37401
12. Corporate Contact: H. G. Parris, Manager of Power
13. Architect/Engineer: TVA
14. Nuclear Steam Supply System: GE
15. Constructor: TVA

Regulatory Information

16. IE Region responsible: II
17. IE Branch Chief: R. C. Lewis
18. IE Section Chief: H. C. Dance
19. IE Resident Inspector: R. Sullivan (205) 729-6196
20. Licensing Project Manager: R. Clark
21. Docket Number: 50-260
22. License Number/Date of Issue: DPR-52, 6/28/74
23. Public Document Room: Athens Public Library
South and Forrest
Athens, Alabama 35611

INSPECTION STATUS

ENFORCEMENT STATUS

TECHNICAL SPECIFICATION 3.11.B.4 REQUIRES IN PART THAT A CONTINUOUS FIRE WATCH SHALL BE ESTABLISHED IMMEDIATELY IF CO2 FIRE PROTECTION IS LOST TO A CABLE SPREADING ROOM. CONTRARY TO THE ABOVE, ON AUGUST 5, 1980 FROM APPROXIMATELY 11:00 A.M. TO 11:45 A.M. A CONTINUOUS FIRE WATCH WAS NOT PROVIDED FOR THE CABLE SPREADING ROOMS WHILE THE CO2 SYSTEM WAS INOPERABLE FOR REPAIRS.
(8027 INFRACTION)

TENNESSEE VALLEY AUTHORITY'S RESPONSE TO IE BULLETIN 80-17, SUPPLEMENT 1, DATED AUGUST 18, 1980 FOR BROWNS FERRY UNIT 1, 2 AND 3, STATES IN PART, THAT THE UT SENSORS ARE CONTINUOUSLY CONNECTED TO A RECORDER WHICH MONITOR FOR AN ACCUMULATION OF WATER IN THE EAST AND WEST SCRAM DISCHARGE HEADERS. THESE RECORDERS ARE CHECKED AT 30 MINUTE INTERVALS. CONTRARY TO THE ABOVE, ON AUGUST 21, 1980, THE RECORDERS WERE NOT CHECKED FROM 11:30 A.M. TO 12:45 P.M. FOR A TOTAL OF ONE HOUR AND 15 MINUTES.
(8027 DEVIATION)

AS REQUIRED BY CRITERION V OF APPENDIX B TO 10CFR50, AND IMPLEMENTED BY TOPICAL REPORT TVA-TR75-01, PARAGRAPH 17.2.5, "ACTIVITIES AFFECTING QUALITY SHALL BE PRESCRIBED BY DOCUMENTED INSTRUCTIONS, PROCEDURES...AND SHALL BE ACCOMPLISHED IN ACCORDANCE WITH THESE INSTRUCTIONS, PROCEDURES AND DRAWINGS". IX-360 OF SECTION III OF THE ASME BOILER AND PRESSURE VESSEL CODE (1968 EDITION) IS IDENTIFIED BY THE FSAR AS THE APPLICABLE CODE FOR INSERVICE INSPECTION PENETRANT EXAMINATION PROCEDURES. IX-360 REQUIRES THAT PENETRANT EXAMINATION PROCEDURES CONTAIN THE DETAILS OF THE METHOD OF PRE-EXAMINATION CLEANING. PENETRANT EXAMINATION PROCEDURE N-PT-1 REVISION 2 REQUIRES REMOVAL OF PAINT, SLAG AND OTHER MASKING MATERIALS FROM THE SPECIMEN SURFACE PRIOR TO PENETRANT APPLICATION. PENETRANT EXAMINATION PROCEDURE N-PT-1 REVISION 2 REQUIRES THAT ALL WELDS BLEND SMOOTHLY INTO THE PARENT METAL. CONTRARY TO THE ABOVE, ON SEPTEMBER 21, 1980, PENETRANT EXAMINATION PROCEDURE N-PT-1 REVISION 2 DID NOT CONTAIN THE DETAILS OF MECHANICAL PRE-EXAMINATION CLEANING METHODS USED TO PREPARE WELDS FOR INSERVICE INSPECTION PENETRANT EXAMINATIONS. ALSO, THE PROCEDURE DID NOT PRESCRIBE THE DETAILS OF THE METHOD TO BE USED TO REMOVE COUPLANT REMAINING ON A WELD FROM PRIOR ULTRASONIC EXAMINATION (WELD TRHR-2-191). PAINT AND SMALL AMOUNTS OF SLAG AND OTHER MASKING MATERIAL WERE NOT REMOVED FROM THE SURFACE OF WELD TRHR-2-191 PRIOR TO APPLICATION OF PENETRANT IN AN INSERVICE INSPECTION PENETRANT EXAMINATION BEING CONDUCTED IN ACCORDANCE WITH PROCEDURE N-PT-1 REVISION 2. WELD TRHR-2-191 WAS NOT PREPARED TO BLEND SMOOTHLY INTO THE PARENT METAL FOR AN INSERVICE INSPECTION PENETRANT EXAMINATION, CONDUCTED IN ACCORDANCE WITH PROCEDURE N-PT-1 REVISION 2.
(8028 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

REACTOR IS IN COLD SHUTDOWN FOR REFUELING

+ LAST IE SITE INSPECTION DATE: SEPTEMBER 16-22, 1980

+ INSPECTION REPORT NO: 50-260/80-28

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-033/ 03L-0	08/15/80	09/08/80	2B RESIDUAL HEAT EXCHANGER LEAKING GASKETED FLANGE.
80-034/ 03L-0	08/15/80	09/08/80	2C RESIDUAL HEAT REMOVAL HEAT EXCHANGER GASKET LEAKAGE.
80-035/ 03L-0	08/30/80	09/12/80	CHECK VALVE SEAT LEAKAGE BETWEEN 2B AND 2D RESIDUAL HEAT REMOVAL HEAT EXCHANGER.
80-036/ 03L-0	08/29/80	09/26/80	HYDROGEN MONITOR WOULD NOT CALIBRATE.

SUMMARY

INSPECTION SUMMARY

+ INSPECTION AUGUST 1-31 (80-27): THIS ROUTINE INSPECTION INVOLVED 43 RESIDENT INSPECTOR-HOURS IN THE AREAS OF OPERATIONAL SAFETY, IE BULLETIN FOLLOWUP, IE CIRCULAR FOLLOWUP, REPORTABLE OCCURRENCES, REACTOR TRIP FOLLOWUP, PLANT PHYSICAL PROTECTION, MAINTENANCE, FIRE PROTECTION, CORE PHYSICS AND ORGANIZATION. OF THE 10 AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE FOUND IN 8 AREAS; ONE APPARENT ITEM OF NONCOMPLIANCE AND ONE APPARENT DEVIATION WAS FOUND IN 2 AREAS (INFRACTION - FAILURE TO PROVIDE A CONTINUOUS FIRE WATCH IN THE CABLE SPREADING ROOMS WITH THE CARBON DIOXIDE SYSTEM INOPERABLE; DEVIATION - FAILURE TO MONITOR SCRAM DISCHARGE HEADER LEVEL EVERY 30 MINUTES AS COMMITTED IN RESPONSE TO IEB 80-17 SUPPLEMENT 1).

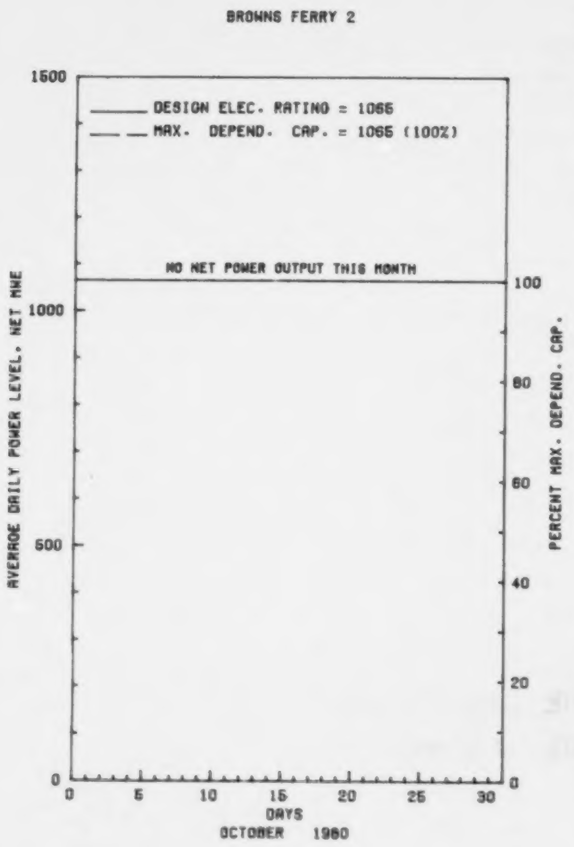
INSPECTION SEPTEMBER 16-22 (80-28): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 51 INSPECTOR-HOURS ON SITE IN THE AREAS OF IE BULLETIN 80-07 AND 80-13, REVIEW OF INSERVICE INSPECTION PROCEDURES, AND OBSERVATION OF INSERVICE INSPECTION WORK. OF THE THREE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED IN ONE AREA; ONE ITEM OF NONCOMPLIANCE WAS FOUND IN TWO AREAS (INFRACTION - INADEQUATE LIQUID PENETRANT EXAMINATION); AND NO DEVIATIONS WERE FOUND.

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 12-10-80

REVIEWED BY NRR DL: Dick Clark **DATE:** 12-10-80

BROWNS FERRY 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-260
2. Utility Contact: Don Green (205) 729-6846
3. Licensed Thermal Power (MWt): 3293
4. Nameplate Rating (Gross MWe): 1280 x 0.9 = 1152
5. Design Electrical Rating (Net MWe): 1065
6. Maximum Dependable Capacity (Gross MWe): 1098
7. Maximum Dependable Capacity (Net MWe): 1065
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	745.0	7,320.0	49,729.0
12. Hours Reactor Was Critical	.0	5,304.1	29,788.5
13. Reactor Reserve Shutdown Hours	.0	649.0	12,423.9
14. Hours Generator On-Line	.0	5,171.1	28,835.6
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	14,929,875	81,084,875
17. Gross Electrical Energy (MWH)	0	4,952,010	26,902,078
18. Net Electrical Energy (MWH)	0	4,807,248	26,138,717
19. Unit Service Factor	.0	70.6	58.0
20. Unit Availability Factor	.0	70.6	58.0
21. Unit Capacity Factor (MDC Net)	.0	61.7	49.4
22. Unit Capacity Factor (DER Net)	.0	61.7	49.4
23. Unit Forced Outage Rate	.0	12.2	33.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 11/07/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
155	09/05/80	S	745.0	C	4				REFUELING OUTAGE CONTINUES.

SUMMARY

BROWNS FERRY 2 REMAINED SHUTDOWN DURING OCTOBER FOR REFUELING.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

BROWNS FERRY 3

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Alabama/Limestone 10 miles NW of Decatur
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	3293
5.	Design Electrical Rating (MWE Net)	1065
6.	Date of Initial Criticality	August 8, 1976
7.	Date of Commercial Operation	March 1, 1977
8.	Condenser Cooling Method	Once Through With Forced Draft Cooling Tower
9.	Condenser Cooling Water Source	Tennessee River

Utility & Contractor Information

10.	Licensee	Tennessee Valley Authority
11.	Corporate Address	500A Chestnut Street Tower II Chattanooga, Tennessee 37401
12.	Corporate Contact	H. G. Parris, Manager of Power
13.	Architect/Engineer	TVA
14.	Nuclear Steam Supply System	GE
15.	Constructor	TVA

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R.C. Lewis
18.	IE Section Chief	H. C. Dance
19.	IE Resident Inspector	R. Sullivan (205) 729-6196
20.	Licensing Project Manager	R. Clark
21.	Docket Number	50-296
22.	License Number/Date of Issue	DPR-68, 7/2/76
23.	Public Document Room	Athens Public Library South and Forrest Athens, Alabama 35611

INSPECTION STATUS

ENFORCEMENT STATUS

TECHNICAL SPECIFICATION 3.11.B.4 REQUIRES IN PART THAT A CONTINUOUS FIRE WATCH SHALL BE ESTABLISHED IMMEDIATELY IF CO2 FIRE PROTECTION IS LOST TO A CABLE SPREADING ROOM. CONTRARY TO THE ABOVE, ON AUGUST 5, 1980 FROM APPROXIMATELY 11:00 A.M. TO 11:45 A.M. A CONTINUOUS FIRE WATCH WAS NOT PROVIDED FOR THE CABLE SPREADING ROOMS WHILE THE CO2 SYSTEM WAS INOPERABLE FOR REPAIRS.
(8028 INFRACTION)

TENNESSEE VALLEY AUTHORITY'S RESPONSE TO IE BULLETIN 80-17, SUPPLEMENT 1, DATED AUGUST 18, 1980 FOR BROWNS FERRY UNIT 1, 2 AND 3, STATES IN PART, THAT THE UT SENSORS ARE CONTINUOUSLY CONNECTED TO A RECORDER WHICH MONITOR FOR AN ACCUMULATION OF WATER IN THE EAST AND WEST SCRAM DISCHARGE HEADERS. THESE RECORDERS ARE CHECKED AT 30 MINUTE INTERVALS. CONTRARY TO THE ABOVE, ON AUGUST 21, 1980, THE RECORDERS WERE NOT CHECKED FROM 11:30 A.M. TO 12:45 P.M. FOR A TOTAL OF ONE HOUR AND 15 MINUTES.
(8028 DEVIATION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

FULL POWER OPERATIONS.

♦ LAST IE SITE INSPECTION DATE: AUGUST 1-31, 1980

♦ INSPECTION REPORT NO: 50-296/80-28

SUMMARY

INSPECTION SUMMARY

♦ INSPECTION AUGUST 1-31 (80-28): THIS ROUTINE INSPECTION INVOLVED 43 RESIDENT INSPECTOR-HOURS IN THE AREAS OF OPERATIONAL SAFETY, IE BULLETIN FOLLOWUP, IE CIRCULAR FOLLOWUP, REPORTABLE OCCURRENCES, REACTOR TRIP FOLLOWUP, PLANT PHYSICAL PROTECTION, MAINTENANCE, FIRE PROTECTION, CORE PHYSICS AND ORGANIZATION. OF THE 10 AREAS INSPECTED, NO ITEMS OF NON-COMPLIANCE OR DEVIATIONS WERE FOUND IN 8 AREAS; ONE APPARENT ITEM OF NONCOMPLIANCE AND ONE APPARENT DEVIATION WAS FOUND IN 2 AREAS (INFRACTION - FAILURE TO PROVIDE A CONTINUOUS FIRE WATCH IN THE CABLE SPREADING ROOMS WITH THE CARBON DIOXIDE SYSTEM INOPERABLE; DEVIATION - FAILURE TO MONITOR SCRAM DISCHARGE HEADER LEVEL EVERY 30 MINUTES AS COMMITTED IN RESPONSE TO IEB 80-17 SUPPLEMENT 1).

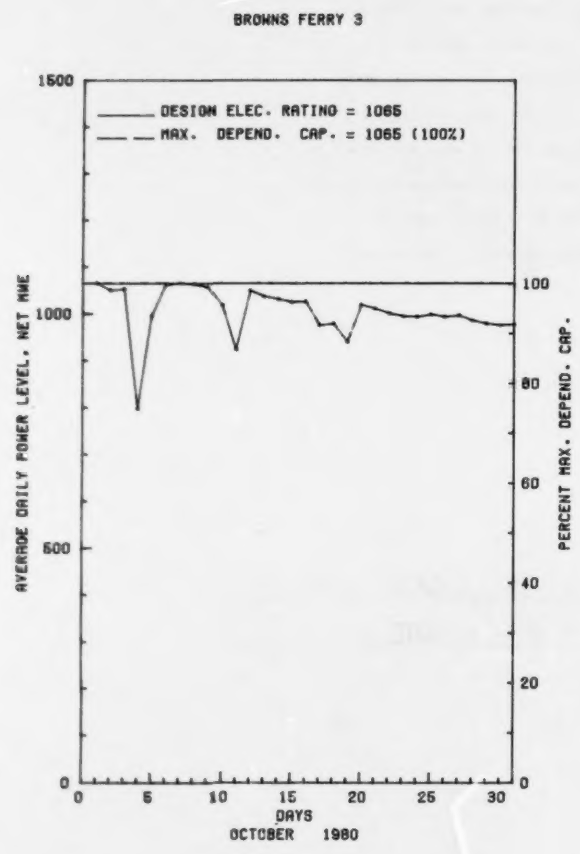
REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-031/ 03L-0	08/09/80	09/08/80	FLOW TO RESIDUAL HEAT REMOVAL PUMP SEAL HEAT EXCHANGER LOW.
80-032/ 03L-0	08/25/80	09/18/80	REACTOR CORE ISOLATION COOLING INOPERABLE.
80-033/ 03L-0	08/15/80	09/12/80	2B RESIDUAL HEAT REMOVAL HEAT EXCHANGER TUBE LEAK.
80-034/ 03L-0	08/29/80	09/19/80	RESIDUAL HEAT REMOVAL AREA COOLER FAN INOPERABLE.
80-035/ 03L-0	09/01/80	09/26/80	"A" REACTOR RECIRCULATION PUMP TRIPPED.
80-036/ 03L-0	09/15/80	09/29/80	5A DIESEL INOPERABLE.
80-037/ 03L-0	09/09/80	09/30/80	HYDROGEN MONITOR INOPERABLE.

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 12-10-80
REVIEWED BY NRR DL: Dick Clark **DATE:** 12-10-80

BROWNS FERRY 3

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-296
2. Utility Contact: Don Green (205) 729-6846
3. Licensed Thermal Power (MWt): 3293
4. Nameplate Rating (Gross MWe): 1280 x 0.9 = 1152
5. Design Electrical Rating (Net MWe): 1065
6. Maximum Dependable Capacity (Gross MWe): 1098
7. Maximum Dependable Capacity (Net MWe): 1065
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	745.0	7,320.0	32,184.0
12. Hours Reactor Was Critical	745.0	6,514.2	25,421.9
13. Reactor Reserve Shutdown Hours	.0	152.6	1,811.0
14. Hours Generator On-Line	745.0	6,402.4	24,840.2
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	2,289,631	19,700,810	72,808,676
17. Gross Electrical Energy (MWH)	769,400	6,612,550	24,015,130
18. Net Electrical Energy (MWH)	748,423	6,429,002	23,316,775
19. Unit Service Factor	100.0	87.5	77.2
20. Unit Availability Factor	100.0	87.5	77.2
21. Unit Capacity Factor (MDC Net)	94.3	82.5	68.0
22. Unit Capacity Factor (DER Net)	94.3	82.5	68.0
23. Unit Forced Outage Rate	.0	12.5	9.9

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
149	10/04/80	F	.0	A	5				"C" REACTOR FEEDWATER PUMP MAINTENANCE (LEAK).
150	10/10/80	S	.0	B	5				BRUSH REPLACEMENT FOR BOTH RECIRCULATION PUMPS MG SET.
151	10/17/80	F	.0	B	5				REPLACE BROKEN BRUSH ON 3B RECIRCULATION PUMP MG SET.
152	10/19/80	F	.0	A	5				"A" AND "B" FLOW BIAS CHANNELS INOPERABLE.

SUMMARY

BROWNS FERRY 3 OPERATED NORMALLY DURING OCTOBER, WITH TWO REDUCTIONS FOR EQUIPMENT FAILURE AND TWO REDUCTIONS FOR MAINTENANCE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

BRUNSWICK 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	North Carolina/Brunswick 3 miles N of Southport
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2436
5.	Design Electrical Rating (MWE Net)	821
6.	Date of Initial Criticality	October 8, 1976
7.	Date of Commercial Operation	March 18, 1977
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Cape Fear River

Utility & Contractor Information

10.	Licensee	Carolina Power and Light Company
11.	Corporate Address	411 Fayetteville Street Raleigh, North Carolina 27602
12.	Corporate Contact	J. A. Jones Executive Vice President
13.	Architect/Engineer	United Engineers & Constructors, Inc.
14.	Nuclear Steam Supply System	GE
15.	Constructor	Brown and Root, Inc.

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R.C. Lewis
18.	IE Section Chief	P.J. Kellogg
19.	IE Resident Inspector	J. E. Outzs (919) 457-9531
20.	Licensing Project Manager	J. Hannon
21.	Docket Number	50-325
22.	License Number/Date of Issue	DPR-71, 9/8/76
23.	Public Document Room	Southport-Brunswick County Library 109 W. Moore Street Southport, North Carolina 28461

INSPECTION STATUS

ENFORCEMENT STATUS

FAILURE TO PERFORM PHYSICAL SEARCH.
(8037 INFRACTION)

FAILURE TO HAVE REQUIRED WRITTEN PROCEDURE.
(8037 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

ROUTINE POWER OPERATION

+ LAST IE SITE INSPECTION DATE: SEPTEMBER 12, 1980

+ INSPECTION REPORT NO: 50-325/80-37

SUMMARY

INSPECTION SUMMARY

+ INSPECTION SEPTEMBER 2-4 (80-36): THIS SPECIAL, UNANNOUNCED INSPECTION INVOLVED 12 INSPECTOR HOURS OF ON SITE TIME IN THE AREAS OF RADIATION DOSIMETRY AND FOLLOWUP OF COMMITMENTS MADE BY PLANT AND CORPORATE MANAGEMENT TO CORRECT PREVIOUSLY IDENTIFIED ITEMS OF NONCOMPLIANCE AND OTHER ITEMS OF CONCERN IN THE BRUNSWICK FACILITIES' RADIATION PROTECTION PROGRAM. DURING THE COURSE OF THE INSPECTION ONE ITEM OF NONCOMPLIANCE WAS IDENTIFIED (INFRACTION - FAILURE TO ADEQUATELY DETERMINE THE WHOLE BODY RADIATION DOSE IN ACCORDANCE WITH 10 CFR 20.201).

INSPECTION SEPTEMBER 8-12 (80-37): INCLUDED REVIEW OF THE LICENSEE'S ACCEPTED SECURITY PLAN; SECURITY PROGRAM AUDIT; SECURITY ORGANIZATION; PROTECTED AREA PHYSICAL BARRIERS; LIGHTING; ACCESS CONTROLS (IDENTIFICATION, BADGING, SEARCH, ESCORTING AND VITAL AREAS); ALARM STATIONS; DETECTION AIDS; ASSESSMENT AIDS; COMMUNICATIONS; LOCKS AND KEYS; RECORDS AND REPORTS. THE INSPECTION INVOLVED 38 INSPECTOR HOURS ON SITE BY TWO NRC INSPECTORS; 6 INSPECTION HOURS WERE ACCOMPLISHED DURING OFFSHIFT PERIODS. OF THE SIXTEEN AREAS INSPECTED, TWO APPARENT ITEMS OF NONCOMPLIANCE WERE IDENTIFIED: (1) ACCESS CONTROL (SEARCH) - FAILURE TO PERFORM PERSONNEL SEARCH; AND (2) SECURITY ORGANIZATION - FAILURE TO PROVIDE A WRITTEN SECURITY PROCEDURE.

INSPECTION SEPTEMBER 4 (80-39): THIS SPECIAL, ANNOUNCED INSPECTION INVOLVED 4 INSPECTOR HOURS AT THE HARRIS ENERGY AND ENVIRONMENTAL CENTER REVIEWING THE CIRCUMSTANCES SURROUNDING RECENT ERRONEOUSLY RECORDED PERSONNEL EXPOSURES FOR THE BRUNSWICK AND H. B. ROBINSON FACILITIES. OF THE ONE AREA INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

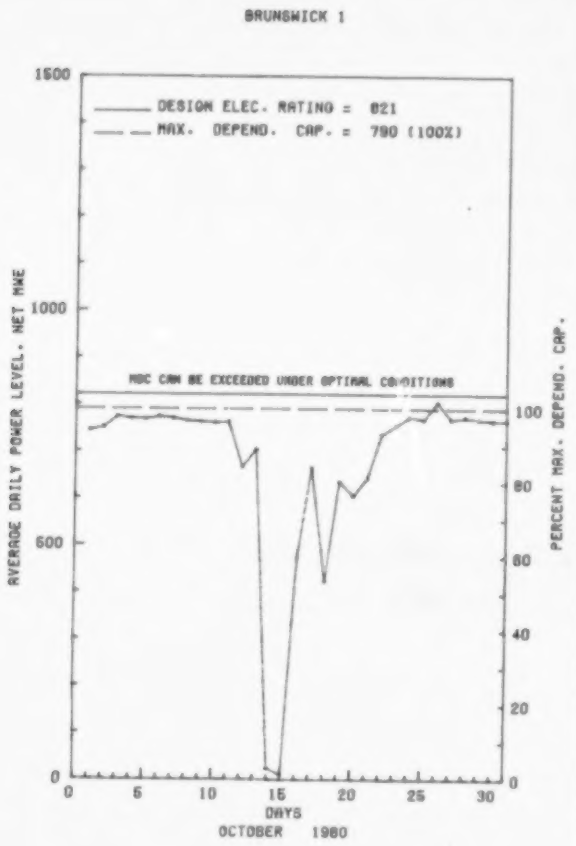
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-17/ 03L-0	09/03/80	09/09/80	RHR HEX 1A OUT OF SERVICE DUE TO PIN-HOLE LEAK IN SERVICE WATER SYSTEM.
80-24/ 03L-0	04/05/80	09/10/80	FAILURE OF DRYWELL EQUIPMENT DRAIN INDICATOR.
80-58/ 03L-0	08/22/80	09/19/80	FAILED DRYWELL FLOOR DRAIN INTEGRATOR.
80-63 03L-0	80-26/80	09/16/80	THREE APRMS INACCURATE DUE TO INSTRUMENT DRIFT.
80-65/ 03L-0	08/24/80	09/16/80	REACTOR VESSEL EXCEEDED CONDUCTIVITY LIMIT.
80-67/ 03L-0	08/22/80	09/16/80	ERRONEOUS ROD BLOCKS FROM ROD WORTH MINIMIZER.
80-69/ 03L-0	08/27/80	09/23/80	RHR FLOW INDICATOR ON REMOTE SHUTDOWN PANEL FAILED.
80-70/ 03L-0	08/28/80	09/24/80	SPURIOUS ALARMS ON NO. 1 DIESEL GENERATOR.
80-71/ 03L-0	09/03/80	09/30/80	CONTAINMENT ATMOSPHERE MONITORS INOPERABLE.

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 12-10-80

REVIEWED BY NRR DL: John N Hannon **DATE:** 12-11-80

BRUNSWICK 1

AVERAGE POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-325
2. Utility Contact: E. Willis (919) 457-9521
3. Licensed Thermal Power (MWt): 2436
4. Nameplate Rating (Gross MWe): 963 x 0.9 = 867
5. Design Electrical Rating (Net MWe): 821
6. Maximum Dependable Capacity (Gross MWe): 815
7. Maximum Dependable Capacity (Net MWe): 790
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	745.0	7,320.0	31,777.0
12. Hours Reactor Was Critical	712.7	5,029.6	22,419.2
13. Reactor Reserve Shutdown Hours	.0	.0	1,647.1
14. Hours Generator On-Line	701.9	4,666.2	20,999.0
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,542,686	9,318,029	43,147,525
17. Gross Electrical Energy (MWH)	517,307	3,115,844	14,354,863
18. Net Electrical Energy (MWH)	502,104	2,998,389	13,806,799
19. Unit Service Factor	94.2	63.7	66.1
20. Unit Availability Factor	94.2	63.7	66.1
21. Unit Capacity Factor (MDC Net)	85.3	51.9	55.0
22. Unit Capacity Factor (DER Net)	82.1	49.9	52.9
23. Unit Forced Outage Rate	5.8	10.4	19.3

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
022	10/14/80	F	43.1	G	3		CC	ZZZZZZ	THE REACTOR SCRAMMED BECAUSE OF HIGH MAIN STEAM LINE FLOW WHILE TESTING THE STOP VALVES. THE HIGH MAIN STEAM LINE FLOW WAS CAUSED BY OPERATING WITH TOO FEW STEAM LINES FOR TOTAL STEAM FLOW DURING MAIN STOP VALVE TESTING. FOLLOWED EMERGENCY INSTRUCTION 31 TO BRING THE PLANT TO A SAFE AND STABLE CONDITION. GENERATED A PROCEDURE FOR PLANT OPERATION WITH ONLY THREE LINES AVAILABLE.
023	10/17/80	F	23.1	B	5		HC	HTEXCH	REDUCED POWER FOR A LEAK CHECK OF 1B NORTH CONDENSER WATERBOX. THE WATERBOX WAS TAKEN OUT OF SERVICE AND LEAK CHECKED USING THE HANDI-WRAP METHOD. NO LEAKS WERE FOUND USING THIS METHOD DUE PROBABLY TO THEIR SMALL SIZE. THE WATERBOX WAS RETURNED TO FULL SERVICE.
024	10/19/80	F	40.0	B	5		HC	HTEXCH	REDUCED POWER TO LEAK CHECK 1B NORTH CONDENSER WATERBOX. THE CONDENSER TUBE LEAK OF OUTAGE 080-022 WHICH WAS TOO SMALL TO BE FOUND USING HANDI-WRAP METHOD WAS TO BE REPAIRED. THE CONTRACTOR FOUND ONE LEAKING TUBE IN THE UPPER RIGHT SECTION. THE INLET AND OUTLET ENDS OF THE TUBE WERE PLUGGED WITH BRASS CONDENSER PLUGS.

SUMMARY

BRUNSWICK 1 OPERATED NORMALLY DURING OCTOBER WITH ONE OUTAGE ON THE 14TH FOR OPERATIONAL ERROR, AND TWO REDUCTIONS ON THE 17TH AND 19TH FOR MAINTENANCE AND TESTING.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

BRUNSWICK 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	North Carolina/Brunswick 3 miles N of Southport
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2436
5.	Design Electrical Rating (MWE Net)	821
6.	Date of Initial Criticality	March 20, 1975
7.	Date of Commercial Operation	November 3, 1975
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Cape Fear River

Utility & Contractor Information

10.	Licensee	Caroline Power and Light Company
11.	Corporate Address	411 Fayetteville Street Raleigh, North Carolina 27602
12.	Corporate Contact	J. A. Jones Executive Vice President
13.	Architect/Engineer	United Engineers & Constructors, Inc.
14.	Nuclear Steam Supply System	GE
15.	Constructor	Brown and Root, Inc.

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R.C. Lewis
18.	IE Section Chief	P.J. Kellogg
19.	IE Resident Inspector	J. E. Outzs (919) 457-9531
20.	Licensing Project Manager	J. Hannon
21.	Docket Number	50-324
22.	License Number/Date of Issue	DPR-62, 12/27/74
23.	Public Document Room	Southport-Brunswick County Library 109 W. Moore Street Southport, North Carolina 28461

INSPECTION STATUS

ENFORCEMENT STATUS

FAILURE TO PERFORM PHYSICAL SEARCH.
(8034 INFRACTION)

FAILURE TO HAVE REQUIRED WRITTEN PROCEDURE.
(8034 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

UNIT IN NORMAL OPERATION.

♦ LAST IE SITE INSPECTION DATE: SEPTEMBER 12, 1980

♦ INSPECTION REPORT NO: 50-324/80-34

SUMMARY

INSPECTION SUMMARY

♦ INSPECTION SEPTEMBER 2-4 (80-33): THIS SPECIAL, UNANNOUNCED INSPECTION INVOLVED 12 INSPECTOR HOURS OF ON SITE TIME IN THE AREAS OF RADIATION DOSIMETRY AND FOLLOWUP OF COMMITMENTS MADE BY PLANT AND CORPORATE MANAGEMENT TO CORRECT PREVIOUSLY IDENTIFIED ITEMS OF NONCOMPLIANCE AND OTHER ITEMS OF CONCERN IN THE BRUNSWICK FACILITIES' RADIATION PROTECTION PROGRAM. DURING THE COURSE OF THE INSPECTION ONE ITEM OF NONCOMPLIANCE WAS IDENTIFIED (INFRACTION - FAILURE TO ADEQUATELY DETERMINE THE WHOLE BODY RADIATION DOSE IN ACCORDANCE WITH 10 CFR 20.201).

INSPECTION SEPTEMBER 8-12 (80-34): INCLUDED REVIEW OF THE LICENSEE'S ACCEPTED SECURITY PLAN; SECURITY PROGRAM AUDIT; SECURITY ORGANIZATION; PROTECTED AREA PHYSICAL BARRIERS; LIGHTING; ACCESS CONTROLS (IDENTIFICATION, BADGING, SEARCH, ESCORTING AND VITAL AREAS); ALARM STATIONS; DETECTION AIDS; ASSESSMENT AIDS; COMMUNICATIONS; LOCKS AND KEYS; RECORDS AND REPORTS. THE INSPECTION INVOLVED 38 INSPECTOR HOURS ON SITE BY TWO NRC INSPECTORS; 6 INSPECTION HOURS WERE ACCOMPLISHED DURING OFF-SHIFT PERIODS. OF THE SIXTEEN AREAS INSPECTED, TWO APPARENT ITEMS OF NONCOMPLIANCE WERE IDENTIFIED: (1) ACCESS CONTROL (SEARCH) - FAILURE TO PERFORM PERSONNEL SEARCH; AND (2) SECURITY ORGANIZATION - FAILURE TO PROVIDE A WRITTEN SECURITY PROCEDURE.

INSPECTION SEPTEMBER 4 (80-36): THIS SPECIAL, ANNOUNCED INSPECTION INVOLVED 4 INSPECTOR HOURS AT THE HARRIS ENERGY AND ENVIRONMENTAL CENTER REVIEWING THE CIRCUMSTANCES SURROUNDING RECENT ERRONEOUSLY RECORDED PERSONNEL EXPOSURES FOR THE BRUNSWICK AND H. B. ROBINSON FACILITIES. OF THE ONE AREA INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATION WERE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

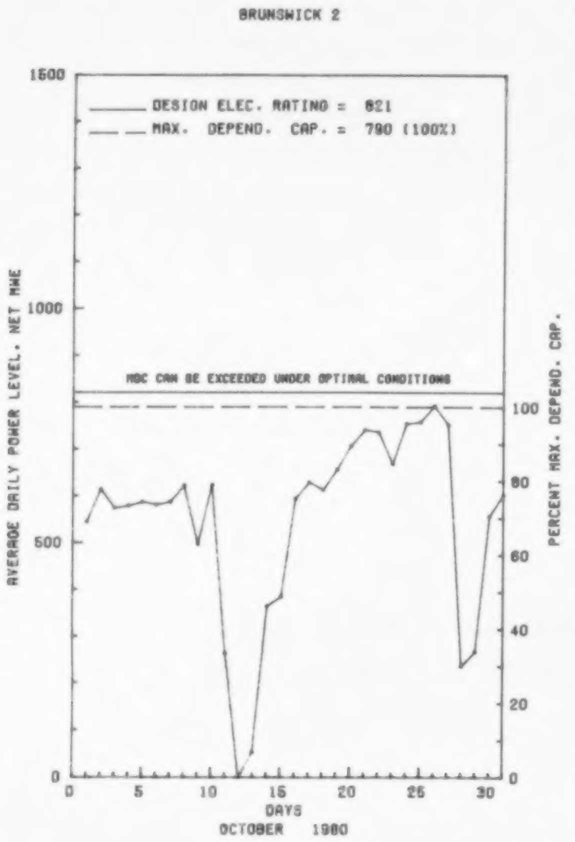
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-052/ 03L-0	09/03/80	10/01/80	INACCURATE CONTROL ROD POSITION INDICATION.
80-060/ 03L-0	09/03/80	09/30/80	CONTROL ROD 38-27 INOPERABLE.
80-062/ 01T-0	09/07/80	09/24/80	STOP CHECK IN TURBINE EXHAUST TO TORUS FOR RCIC AND HPIC CLOSED.
80-064/ 03L-0	09/05/80	10/03/80	GROUND ON RX LEVEL SWITCH.
80-065/ 03L-0	09/06/80	09/30/80	CONTAINMENT ATMOSPHERE MONITOR 2 CAC AT 1259 INOPERABLE.
80-066/ 01T-0	09/07/80	09/19/80	STOP CHECK IN TURBINE EXHAUST TO TORUS FOR RCIC AND HPIC CLOSED.
80-066/ 03L-0	09/05/80	10/03/80	LOOP A OF RHR SERVICE WATER INOPERABLE.
80-075/ 03L-0	09/03/80	09/23/80	SQUARE ROOT TRANSMITTER 2E11-FY-3338 FOUND NOT RESPONDING PROPERLY (RHR SYSTEM).

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 12-10-80

REVIEWED BY NRR DL: **DATE:** 12-11-80

BRUNSWICK 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-324
2. Utility Contact: E. Willis (919) 457-9521
3. Licensed Thermal Power (MWt): 2436
4. Nameplate Rating (Gross MWe): 963 x 0.9 = 867
5. Design Electrical Rating (Net MWe): 821
6. Maximum Dependable Capacity (Gross MWe): 815
7. Maximum Dependable Capacity (Net MWe): 790
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	745.0	7,320.0	43,801.0
12. Hours Reactor Was Critical	686.3	2,537.8	27,861.7
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	668.0	2,074.2	25,948.2
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,261,271	4,016,802	47,751,264
17. Gross Electrical Energy (MWH)	423,582	1,346,880	15,961,954
18. Net Electrical Energy (MWH)	406,359	1,288,333	15,328,805
19. Unit Service Factor	89.7	28.3	59.2
20. Unit Availability Factor	89.7	28.3	59.2
21. Unit Capacity Factor (MDC Net)	69.0	22.3	44.3
22. Unit Capacity Factor (DER Net)	66.4	21.4	42.6
23. Unit Forced Outage Rate	10.3	13.8	13.1

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
010	10/11/80	F	54.2	G	3		ZZ	ZZZZZZ	REACTOR SCRAMMED WHILE I&C TECHNICIANS WERE PERFORMING PT 1.3.2P. THE APRM SETDOWN WAS NOT RESET AFTER COMPLETING CHANNEL A OF PT 1.3.2P. RESTARTED REACTOR. REVISED PT 1.3.2P TO HAVE OPERATOR RESET APRM SETDOWN.
011	10/28/80	F	22.8	A	3		CH	CKTBRK	HIGH REACTOR WATER LEVEL CAUSED A TRIP OF THE MAIN TURBINE RESULTING IN A REACTOR SCRAM, APPARENTLY A PROBLEM IN THE FEEDWATER CONTROL SYSTEM. THE TOPAZ INVERTER IN THE FEEDWATER LEVEL CONTROL CIRCUIT TRIPPED ON HIGH VOLTAGE. THE HIGH VOLTAGE TRIP POINT WAS RESET TO 147 VDC. THE OTHER TOPAZ INVERTER TRIP POINTS WERE CHECKED AND ADJUSTED AS NECESSARY PRIOR TO RESTART OF THE UNIT.

SUMMARY

BRUNSWICK NO. 2 OPERATED AT A 69.0% CAPACITY FACTOR FOR THE MONTH OF OCTOBER WITH A 89.7% AVAILABILITY FACTOR. THE UNIT WAS "DOWN" FOR 77 HOURS. THE TWO OUTAGES WERE DUE TO EQUIPMENT FAILURE AND OPERATIONAL ERROR.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Continuations
5-Load Reduction
9-Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensed Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

CALVERT CLIFFS 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Maryland/Calvert 40 miles S of Annapolis
2.	Electric Reliability Council	Mid-Atlantic Area Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2700
5.	Design Electrical Rating (MWE Net)	845
6.	Date of Initial Criticality	October 7, 1974
7.	Date of Commercial Operation	May 8, 1975
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Chesapeake Bay

Utility & Contractor Information

10.	Licensee	Baltimore Gas and Electric Company
11.	Corporate Address	P. O. Box 1475 Baltimore, Maryland 21203
12.	Corporate Contact	A. E. Lundvall, Jr. Vice President-Supply
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Combustion Engineering
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Section Chief	E. C. McCabe, Jr.
19.	IE Resident Inspector	R. Architzel (301) 586-2627
20.	Licensing Project Manager	E. L. Conner, Jr.
21.	Docket Number	50-317
22.	License Number/Date of Issue	DPR-53, 7/31/74
23.	Public Document Room	Calvert County Library Prince Frederick, Maryland 20678

INSPECTION STATUS

ENFORCEMENT STATUS

```
CONTRARY TO TECHNICAL SPECIFICATION 6.8.1: PLANT PROCEDURE CCI 2
107D AND 214A - FAILURE TO OBTAIN AGI REVIEW OF COMPLETED MAINTE 018,
PROCEDURES; COMPLETED MAINTENANCE PROCEDURES AND CHECKOFF LISTS NANCE
NOT FILED WITH THE ASSOCIATED MRS.                              WERE
(8013 DEFICIENCY)

CONTRARY TO TECHNICAL SPECIFICATIONS 6.8.1; ANSI N18.7-1972, PAR
5.12 AND 5.4 - ALARM PROCEDURES WERE MAINTAINED INCONSISTENT TO AGRAPHS
THE OPERATORS WERE NOT AWARE OF ALARM MODIFICATION AND PROCEDURE ALARMS;
CHANGES; SUPERSEDED PROCEDURES WERE NOT REMOVED FROM THE CONTROL
FILES.                                                          LED
(8013 DEFICIENCY)

CONTRARY TO 10 CFR 50, APPENDIX B, CRITERION V -  ALARM MODIFICA
AND PROCEDURE CHANGES NOT INCORPORATED INTO THE PLANT EMERGENCY TION
PROCEDURE; ALARM PROCEDURES NOT REVIEWED PROPERLY FOR ADEQUACY.
(8013 DEFICIENCY)
```

```
     INSPECTION  STATUS

     OTHER ITEMS OF INTEREST

     PLANT STATUS:

+    REFUELING, TMI-LL MODIFICATIONS, AND MAINTENANCE
     OUTAGE BEGAN ON 10/17/80.

+ LAST IE SITE INSPECTION DATE:  10/27-30/80

+          INSPECTION REPORT NO:  50-317/80-20

     SUMMARY

     INSPECTION SUMMARY

+    500317/80-13 - JUL 28 - AUG 1: ROUTINE, UNANNOUNCED INSPEC
     BY TWO REGION-BASED INSPECTORS (42 HRS) OF: PREVIOUSLY IDEIION
     FIED ITEMS; ADMINISTRATIVE CONTROLS FOR FACILITY PROCEDURESTI-
     CONFORMANCE TO TECHNICAL SPECIFICATIONS; TEMPORARY AND PER);
     MANENT PROCEDURE CHANGE CONFORMANCE TO TECHNICAL SPECIFICA
     AND LICENSEE PROCEDURES; PROCEDURAL CHANGE CONFIRMANCE TO TIONS
     59(A) AND (B); CHECKLISTS AND RELATED FORMS INCORPORATION 60.
     LATEST CHANGES; ADMINISTRATIVE CONTROLS FOR SAFETY-RELATED)F
     MAINTENANCE: SAFETY-RELATED MAINTENANCE ACTIVITIES; MAINTE
     PERSONNEL QUALIFICATIONS; AND CONTROL ROOM AND FACILITY TOWANCE
     THREE ITEMS OF NONCOMPLIANCE WERE FOUND (FAILURE TO MAINTA)RS.
     ALARM PROCEDURES CONSISTENT WITH THE INDICATED ALARM STATUIN
     AND TO RECOGNIZE THE CHANGES IN ALARM SETPOINT AND PROCEDUR.
     INAPPROPRIATE PROCEDURES FOR ACTUAL CONDITIONS, IN THAT PRIE;
     CHANGES TO THE SECONDARY CEA POSITION DEVIATION ALARM SETP(OR
     STATUS ALARM WINDOW AND CORRECT SETPOINTS FOR THE 11B REAC)INT,
     COOLANT PUMP SEAL TEMPERATURE HIGH PRESSURE WERE NOT REFLE(OR
     IN PROCEDURES, FAILURE TO PROPERLY MAINTAIN AND REVIEW MAINTED
     TENANCE RECORDS).                                          I-

+    50-317/80-14 - AUG 1-31: ROUTINE, ONSITE REGULAR AND BACKSH
     INSPECTION BY THE RESIDENT INSPECTOR (22 HRS) AND ONE REGIOIFT
     BASED INSPECTOR (10 HRS).  AREAS INSPECTED INCLUDED: THE CON-
     ROOM AND THE ACCESSIBLE PORTIONS OF THE AUXILIARY, TURBINE)NTROL
     SERVICE, AND INTAKE BUILDINGS; RADIATION PROTECTION, PHYSI(
     SECURITY; FIRE PROTECTION; PLANT OPERATING RECORDS; IE BULLAL
     AND CIRCULARS; EVENTS REQUIRING REPORTING; LICENSE AMENDMEIETINS
     IMPLEMENTATION; REVIEW OF LOAD CENTERS; AND REPORTS TO THE T
     NO NONCOMPLIANCES WERE IDENTIFIED.                         NRC.
-
+    50-317/80-17 - SEP 16-19: ROUTINE UNANNOUNCED INSPECTION BY
     REGION-BASED INSPECTOR (29 HRS) AND THE SENIOR RESIDENT    A
     INSPECTOR (4 HRS) OF QUALITY ASSURANCE PROGRAM (QAP) IMPLE-
     MENTATION INCLUDING: AUDITING; DESIGN CHANGES/MODIFICATIONS
     QAP CHANGES, AND FOLLOWUP ON PRIOR IDENTIFIED ITEMS.  NO NO
     COMPLIANCES WERE IDENTIFIED.                               N-
```

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

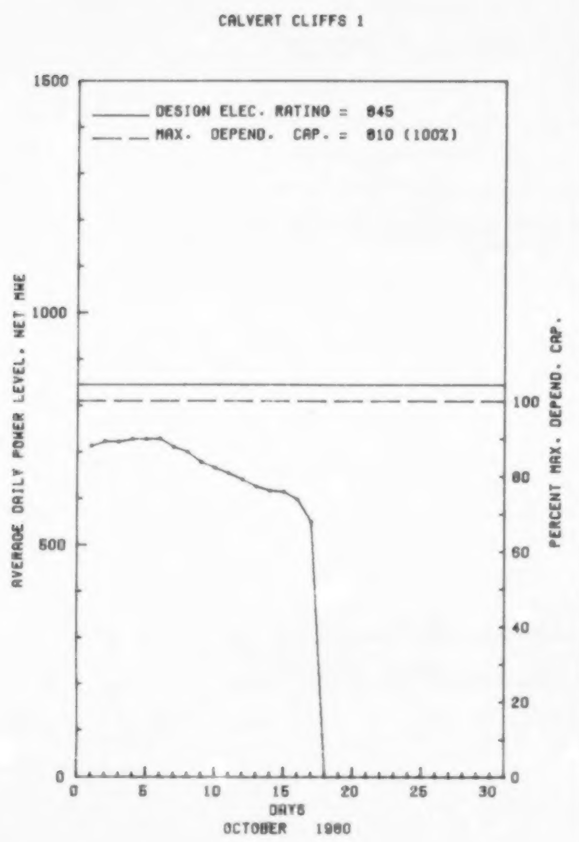
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-047/03L	09/20/80	10/17/80	AFTER COMPLETION OF NORMAL CONTAINMENT DRAINING, SUMP DRAIN VALVE FAILED TO SHUT.
80-049/03L	08/30/80	09/26/80	NO. 12 CONTROL ROOM AIR CONDITIONING UNIT INOPERABLE.
80-050/03L	09/01/80	10/01/80	12A SAFETY INJECTION TANK LEVEL INCREASED FROM 199 TO 245 INCHES DURING FILLING - APPARENTLY CAUSED BY A STICKY TRANSMITTER.
80-051/01T	09/11/80	09/24/80	BOTH CONTROL ROOM AIR CONDITIONING UNITS FAILED WHEN #11 UNIT WAS DOWN FOR POST MAINTENANCE TESTING. #12 A/C FAILED TO RESTART.
80-052/03L	09/17/80	10/17/80	DURING NORMAL POWER OPERATION #12 SALTWATER SERVICE WATER HEAT EXCHANGER WAS OUT OF SERVICE TO REPAIR TUBE LEAK.
80-053/01T	09/26/80	10/03/80	DURING REVIEW OF IE BULLETIN 80-11 THREE STACKED SHIELDING WALLS WERE IDENTIFIED AS POSSIBLY IMPACTING ON SAFETY-RELATED SYSTEM.S

REVIEWED BY OMPA LOEB: MR Beebe **DATE:** 12/10/80

REVIEWED BY NRR DL: E L Conner **DATE:** 12/11/80

CALVERT CLIFFS 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-317
2. Utility Contact: S. D. Merson (301) 234-5248
3. Licensed Thermal Power (MWt): 2700
4. Nameplate Rating (Gross MWe): 1020 x 0.9 = 918
5. Design Electrical Rating (Net MWe): 845
6. Maximum Dependable Capacity (Gross MWe): 845
7. Maximum Dependable Capacity (Net MWe): 810
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	745.0	7,320.0	48,085.0
12. Hours Reactor Was Critical	410.1	6,421.1	38,860.7
13. Reactor Reserve Shutdown Hours	.0	38.3	1,098.8
14. Hours Generator On-Line	409.1	6,350.7	38,052.7
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	923,611	15,229,804	90,566,792
17. Gross Electrical Energy (MWH)	288,487	4,781,783	29,594,233
18. Net Electrical Energy (MWH)	271,772	4,543,042	28,192,248
19. Unit Service Factor	54.9	86.8	79.1
20. Unit Availability Factor	54.9	86.8	79.1
21. Unit Capacity Factor (MDC Net)	45.0	76.6	72.4
22. Unit Capacity Factor (DER Net)	43.2	73.4	69.4
23. Unit Forced Outage Rate	.0	2.9	8.0

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 12/2/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-11	10/18/80	S	335.9	C	1		XX	ZZZZZZ	REFUELING, UNIT GENERAL INSPECTION AND TMI MODIFICATIONS.

SUMMARY

CALVERT CLIFFS 1 OPERATED NORMALLY DURING OCTOBER THRU THE 17TH, AND WENT INTO A REFUELING OUTAGE ON THE 18TH.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

CALVERT CLIFFS 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1. Location (state/county) (nearest population center)	Maryland/Calvert 40 miles S of Annapolis
2. Electric Reliability Council	Mid-Atlantic Area Council
3. Type of Reactor	PWR
4. Power Level (MWT)	2700
5. Design Electrical Rating (MWE Net)	845
6. Date of Initial Criticality	November 30, 1976
7. Date of Commercial Operation	April 1, 1977
8. Condenser Cooling Method	Once Through
9. Condenser Cooling Water Source	Chesapeake Bay

Utility & Contractor Information

10. Licensee	Baltimore Gas and Electric Company
11. Corporate Address	P. O. Box 1475 Baltimore, Maryland 21203
12. Corporate Contact	A. E. Lundvall, Jr. Vice President-Supply
13. Architect/Engineer	Bechtel
14. Nuclear Steam Supply System	Combustion Engineering
15. Constructor	Bechtel

Regulatory Information

16. IE Region responsible	I
17. IE Branch Chief	E. Brunner
18. I E Section Chief	E. C. MCCABE, JR.
19. IE Resident Inspector	R. Architzel (301) 586-2627
20. Licensing Project Manager	E. L. Conner, Jr.
21. Docket Number	50-318
22. License Number/Date of Issue	DPR-69, 8/13/76
23. Public Document Room	Calvert County Library Prince Frederick, Maryland 20678

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO TECHNICAL SPECIFICATIONS 6.8.1; PLANT PROCEDURES CCI201B, 107D AND 214A - FAILURE TO OBTAIN AGI REVIEW OF COMPLETED MAINTENANCE PROCEDURES; COMPLETED MAINTENANCE PROCEDURES AND CHECKOFF LISTS WERE NOT FILED WITH THE ASSOCIATED MRS.
(8012 DEFICIENCY)

CONTRARY TO TECHNICAL SPECIFICATIONS 6.8.1; ANSI N18.7-1972, PARAGRAPHS 5.12 AND 5.4, ALARM PROCEDURES WERE MAINTAINED INCONSISTENT TO ALARMS; THE OPERATORS WERE NOT AWARE OF ALARM MODIFICATION AND PROCEDURE CHANGES; SUPERSEDED PROCEDURES WERE NOT REMOVED FROM THE CONTROLLED FILES.
(8012 DEFICIENCY)

CONTRARY TO TECHNICAL SPECIFICATIONS 6.8.2; ANSI N18.7-1972, PARAGRAPH 5.1 - ALARM MODIFICATIONS AND PROCEDURE CHANGES NOT INCORPORATED INTO THE PLANT EMERGENCY PROCEDURE; ALARM PROCEDURES NOT REVIEWED PROPERLY FOR ADEQUACY.
(8012 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS:

♦ ROUTINE OPERATION.

♦ LAST IE SITE INSPECTION DATE: 10/27-30/80

♦ INSPECTION REPORT NO: 50-318/80-17

SUMMARY

INSPECTION SUMMARY

♦ 50-318/80-12 - JUL 28 - AUG 1: ROUTINE, UNANNOUNCED INSPECTION BY TWO REGION-BASED INSPECTORS (20 HRS) OF: PREVIOUSLY IDENTIFIED ITEMS; ADMINISTRATIVE CONTROLS FOR FACILITY PROCEDURES; CONFORMANCE TO TECHNICAL SPECIFICATIONS; TEMPORARY AND PERMANENT PROCEDURE CHANGE CONFORMANCE TO TECHNICAL SPECIFICATION REQUIREMENTS AND LICENSEE PROCEDURES; PROCEDURAL CHANGE CONFORMANCE TO 50.59( ) AND (B) REQUIREMENTS; CHECKLISTS AND RELATED FORMS INCORPORATION OF LATEST CHANGES; ADMINISTRATIVE CONTROLS FOR SAFETY-RELATED MAINTENANCE; SAFETY-RELATED MAINTENANCE ACTIVITIES; MAINTENANCE PERSONNEL QUALIFICATIONS; AND CONTROL ROOM AND FACILITY TOURS. THREE ITEMS OF NONCOMPLIANCE WERE FOUND: (FAILURE TO MAINTAIN ALARM PROCEDURES CONSISTENT WITH THE INDICATED ALARM STATUS, AND TO RECOGNIZE THE CHANGES IN ALARM SETPOINT AND PROCEDURE BY THE OPERATORS; INAPPROPRIATE PROCEDURES FOR ACTUAL CONDITIONS, IN THAT PRIOR CHANGES TO THE SECONDARY CEA POSITION DEVIATION ALARM SETPOINT, STATUS ALARM WINDOW AND CORRECT SETPOINTS FOR THE 11B REACTOR COOLANT PUMP SEAL TEMPERATURE HIGH PRESSURE WERE NOT REFLECTED IN PROCEDURES; FAILURE TO MAINTAIN AND REVIEW MAINTENANCE RECORDS PROPERLY).

♦ 50-318/80-13 - AUG 1-31: ROUTINE, ONSITE REGULAR AND BACKSHIFT INSPECTION BY THE RESIDENT INSPECTOR (22 HRS). AREAS INSPECTED INCLUDED: THE CONTROL ROOM AND THE ACCESSIBLE PORTIONS OF THE AUXILIARY, TURBINE, SERVICE, AND INTAKE BUILDINGS; RADIATION PROTECTION; PHYSICAL SECURITY; FIRE PROTECTION; PLANT OPERATING RECORDS; IE BULLETINS AND CIRCULARS; EVENTS REQUIRING REPORTING; LICENSE AMENDMENT IMPLEMENTATION; REVIEW OF LOAD CENTERS; AND REPORTS TO THE NRC. NO NONCOMPLIANCES WERE IDENTIFIED.

♦ 50-318/80-16 - SEP 16-19: ROUTINE UNANNOUNCED INSPECTION BY A REGION-BASED INSPECTOR (29 HRS) AND THE SENIOR RESIDENT INSPECTOR (4 HRS) OF QUALITY ASSURANCE PROGRAM (QAP) IMPLEMENTATION INCLUDING: AUDITING; DESIGN CHANGES/MODIFICATIONS; QAP CHANGES; AND FOLLOWUP ON PRIOR IDENTIFIED ITEMS. NO NONCOMPLIANCES WERE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

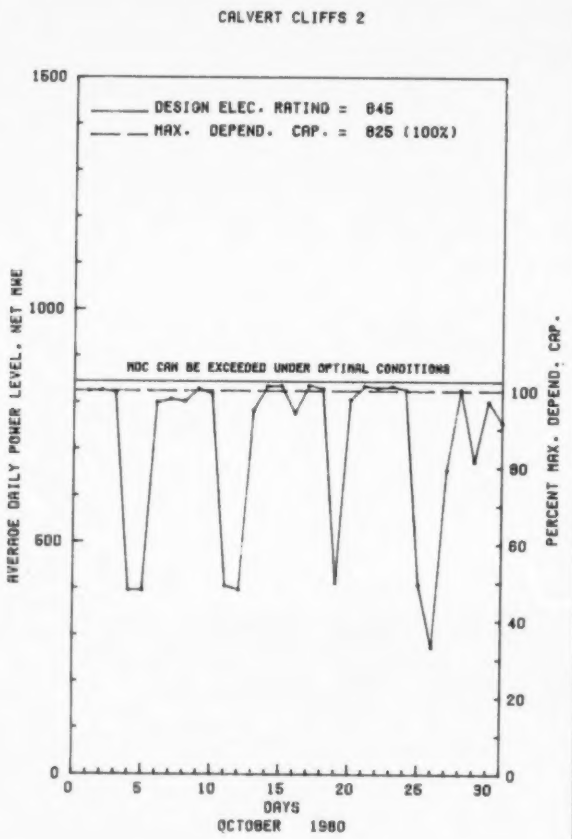
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-037/ 03L	09/10/80	09/26/80	CONTAINMENT SUMP DRAIN VALVE LEFT OPEN FOR 30 MINUTES. ONE METHOD OF RCS LEAKAGE DETERMINATION INOPERABLE.
80-038/ 03X	08/18/80	10/14/80	STEAM GENERATOR BLOWDOWN PH BELOW SPECIFICATION FOR 2 HOURS CAUSED BY TECHNICIAN FAILURE TO ASSURE PROPER CHEMICAL ADDITIONAL LINEUP. (SUBMITTED EARLIER AS 80-37/3L)
80-039/ 03L	08/29/80	09/29/80	CHANNEL C PRESSURIZER PRESSURE TRIP OUT OF SPECIFICATION HIGH DUE TO SETPOINT DRIFT
80-040/ 03L	09/01/80	10/01/80	CONTROL ELEMENT ASSEMBLY NO. 34 DROPPED INTO CORE REQUIRING A POWER REDUCTION
80-041/ 03L	09/15/80	10/15/80	CEA 09 DROPPED INTO CORE DURING PREVENTIVE MAINTENANCE PROCEDURE.
80-042/ 03L	09/15/80	10/10/80	DURING SURVEILLANCE TEST SAFETY INJECTION SYSTEM HYDRAULIC SNUBBERS WERE REPLACED WITH NEW ONES.
80-043/ 03L	09/14/80	10/14/80	22B SIT LEVEL INDICATION BECAME ERRATIC AND INCREASED TO GREATER THAN 199". REACTOR WAS PLACED IN MODE 3 AND PRESSURE REDUCED. LEVEL TRANSMITTER WAS REPLACED AND RECALIBRATED.

REVIEWED BY OMPA LOEB: MR Beebe **DATE:** 12/10/80

REVIEWED BY NRR DL: E L Conner **DATE:** 12/11/80

CALVERT CLIFFS 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-318
2. Utility Contact: S. D. Merson (301) 234-5248
3. Licensed Thermal Power (MWt): 2700
4. Nameplate Rating (Gross MWe): 1012 x 0.9 = 911
5. Design Electrical Rating (Net MWe): 845
6. Maximum Dependable Capacity (Gross MWe): 860
7. Maximum Dependable Capacity (Net MWe): 825
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

ITEM 21 CALCULATED WITH A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	745.0	7,320.0	31,440.0
12. Hours Reactor Was Critical	738.8	7,010.9	26,955.8
13. Reactor Reserve Shutdown Hours	6.2	28.7	441.8
14. Hours Generator On-Line	736.2	6,964.6	26,573.1
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,725,686	17,445,418	65,689,649
17. Gross Electrical Energy (MWH)	553,654	5,718,241	21,723,464
18. Net Electrical Energy (MWH)	527,038	5,458,874	20,715,894
19. Unit Service Factor	98.8	95.1	84.5
20. Unit Availability Factor	98.8	95.1	84.5
21. Unit Capacity Factor (MDC Net)	85.7	90.4	80.9
22. Unit Capacity Factor (DER Net)	83.7	88.3	78.0
23. Unit Forced Outage Rate	1.2	1.3	5.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
REFUELING, INSPECTION & TMI MODS. - 01/02/81 THRU 03/02/81.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-11	10/04/80	S	51.0	H	5		XX	ZZZZZZ	FUEL CONSERVATION, CAPACITY AVAILABLE ON DEMAND.
80-12	10/11/80	S	50.0	H	5		XX	ZZZZZZ	FUEL CONSERVATION, CAPACITY AVAILABLE ON DEMAND.
80-13	10/19/80	S	31.0	H	5		XX	ZZZZZZ	FUEL CONSERVATION, CAPACITY AVAILABLE ON DEMAND.
80-14	10/26/80	F	8.8	A	2		XX	PUMPXX	LOSS OF CONDENSER VACUUM WHEN 22 CONDENSER AIR REMOVAL PUMP FAILED.

SUMMARY

CALVERT CLIFFS 2 OPERATED NORMALLY DURING OCTOBER, REDUCING LOAD ON WEEKENDS TO CONSERVE FUEL, AND HAD ONE OUTAGE ON THE 26TH FOR EQUIPMENT FAILURE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

NRC POWER RESTRICTIONS: NONE

COOK 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Michigan/Berrien 11 miles S of Benton Harbor
2.	Electric Reliability Council	East Central Area Reliability Coordination Agreement
3.	Type of Reactor	PWR
4.	Power Level (MWT)	3250
5.	Design electrical Rating (MWE Net)	1054
6.	Date of Initial Criticality	January 18, 1975
7.	Date of Commercial Operation	August 27, 1975
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10.	Licensee	AEP Corporation
11.	Corporate Address	Indiana & Michigan Electric Co. 2 Broadway New York, New York 10004
12.	Corporate Contact	John Dolan Senior Executive V. P. Eng.
13.	Architect/Engineer	AEP
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Indiana & Michigan Power Co.

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	D.C. Boyd
19.	IE Resident Inspector	E. R. Swanson (616) 465-6775
20.	Licensing Project Manager	S. Miner
21.	Docket Number	50-315
22.	License Number/Date of Issue	DPR-58 issued 10/25/74
23.	Public Document Room	Maude Preston Palenske Memorial Library 500 Market Street St. Joseph, Michigan 49085

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

FACILITY ITEMS

+ ADDED AN AUXILIARY MOTOR DRIVEN FEED PUMP TO EACH UNIT. (80-13)

PLANT STATUS

OPERATING ROUTINELY.

+LAST IE SITE INSPECTION DATE: JUNE 16 THROUGH SEPTEMBER 27, 1980

+ INSPECTION REPORT NO: 80-13

SUMMARY

INSPECTION SUMMARY

+ INSPECTION ON JUNE 16 THROUGH SEPTEMBER 27, (80-13): OPERATIONAL SAFETY VERIFICATION, SURVEILLANCE OBSERVATION, SAFETY SYSTEM CHALLENGES, REFUELING ACTIVITIES, DESIGN CHANGES AND MODIFICATIONS. THE INSPECTION INVOLVED 306 INSPECTOR-HOURS ONSITE BY THREE NRC INSPECTORS INCLUDING 48 INSPECTOR-HOURS ONSITE DURING OFF-SHIFTS. OF THE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80 21/ 03L-0	09/17/80	10/16/80	TWENTY-ONE INTERMEDIATE DECK DOORS WERE FOUND FROZEN AND DECLARED INOPERABLE.
SI-04		09/26/80	AT 44 MINUTES INTO COOLDOWN AND DEPRESS. PROCEDURE, SAFETY INJECTION WAS INITIATED FROM LOW PRESSURIZER PRESSURE. (SPECIAL REPORT)

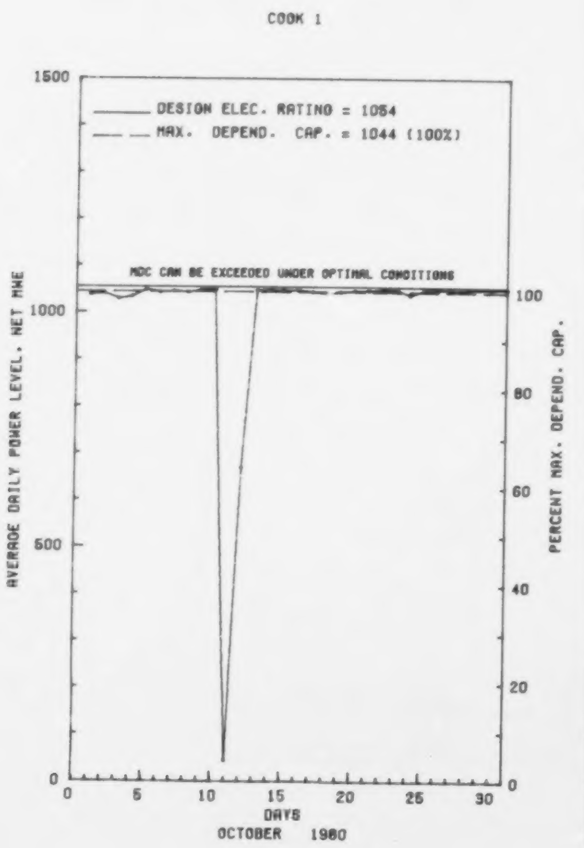
==

REVIEWED BY OMPA LOEB: PC Bobe **DATE:** 12/10/80

REVIEWED BY NRR DL: Sydney Miner **DATE:** 12/10/80

COOK 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-315

2. Utility Contact: W. T. Gillett (616) 465-5901

3. Licensed Thermal Power (MWt): 3250

4. Nameplate Rating (Gross MWe): 1280 x 0.9 = 1152

5. Design Electrical Rating (Net MWe): 1054

6. Maximum Dependable Capacity (Gross MWe): 1080

7. Maximum Dependable Capacity (Net MWe): 1044

8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE

9. Power Level To Which Restricted, If Any (Net MWe): NONE

10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	745.0	7,320.0	51,144.0
12. Hours Reactor Was Critical	724.0	5,272.3	38,234.0
13. Reactor Reserve Shutdown Hours	.0	.0	463.0
14. Hours Generator On-Line	721.0	5,187.0	37,270.2
15. Unit Reserve Shutdown Hours	.0	.0	321.0
16. Gross Thermal Energy (MWH)	2,312,061	16,033,104	105,899,489
17. Gross Electrical Energy (MWH)	771,050	5,316,090	34,473,420
18. Net Electrical Energy (MWH)	744,589	5,126,261	33,123,565
19. Unit Service Factor	96.8	70.9	76.1
20. Unit Availability Factor	96.8	70.9	76.1
21. Unit Capacity Factor (MDC Net)	99.6	67.5	64.0
22. Unit Capacity Factor (DER Net)	98.6	66.8	62.7
23. Unit Forced Outage Rate	3.2	8.9	7.1

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE

25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
168	10/11/80	F	24.0	B	3		ZZ	ZZZZZZ	WHILE ATTEMPTING TO PERFORM TURBINE CONTROL VALVE TESTING AT 100% POWER, CONTROL VALVE OSCILLATIONS BECAME EXCESSIVE. THE OSCILLATIONS CREATED STEAM GENERATOR PRESSURE FLUCTUATIONS WHICH RESULTED IN "HIGH DIFFERENTIAL PRESSURE BETWEEN STEAM LEADS," CAUSING SAFETY INJECTION ACTUATION TO OCCUR. THE SAFETY INJECTION SIGNAL CAUSED THE REACTOR/TURBINE TRIP. THE SAFETY INJECTION WAS RESET AND THE PUMPS SHUTDOWN AFTER 12 MINUTES OF OPERATION.

SUMMARY

COOK 1 OPERATED AT OR NEAR FULL POWER FOR MOST OF OCTOBER WITH ONE OUTAGE ON THE 11TH FOR TESTING.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

COOK 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Michigan/Berrien 11 miles S of Benton Harbor
2.	Electric Reliability Council	East Central Area Reliability Coordination Agreement
3.	Type of Reactor	PWR
4.	Power Level (MWT)	3391
5.	Design Electrical Rating (MWE Net)	1100
6.	Date of Initial Criticality	March 10, 1978
7.	Date of Commercial Operation	July 1, 1978
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10.	Licensee	AEP Corporation Indiana & Michigan Electric Co.
11.	Corporate Address	2 Broadway New York, New York 10004
12.	Corporate Contact	John Dolan, VP/Chief Engineer (212) 422 - 4800
13.	Architect/Engineer	American Elec. Power Service Corp.
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Indiana & Michigan Power Co.

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	D.C. Boyd
19.	IE Resident Inspector	E. Swanson (616) 465-6775
20.	Licensing Project Manager	S. Miner
21.	Docket Number	50-316
22.	License Number/Date of Issue	DPR-74, 12/23/77
23.	Public Document Room	Maude Preston Palenske Memorial Library 500 Market Street St. Joseph, Michigan 49085

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

FACILITY ITEMS

- ADDED AN AUXILIARY MOTOR DRIVEN FEED PUMP TO EACH UNIT. (80-11)

PLANT STATUS

- UNIT SHUTDOWN ON OCTOBER 18, 1980 DUE TO FAULT IN MAIN GENERATOR. REPAIRS ESTIMATED TO REQUIRE 8 WEEKS.

*LAST IE SITE INSPECTION DATE: AUGUST 27-28, 1980

- INSPECTION REPORT NO: 80-12

SUMMARY

INSPECTION SUMMARY

- INSPECTION ON JUNE 16 THROUGH SEPTEMBER 27, (80-11): OPERATIONAL SAFETY VERIFICATION, SURVEILLANCE OBSERVATION, SAFETY SYSTEM CHALLENGES, REFUELING ACTIVITIES, DESIGN CHANGES AND MODIFICATIONS. THE INSPECTION INVOLVED 306 INSPECTOR-HOURS ONSITE BY THREE NRC INSPECTORS INCLUDING 48 INSPECTOR-HOURS ONSITE DURING OFF-SHIFTS. OF THE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.
- INSPECTION ON AUGUST 27-28, (80-12): ROUTINE, ANNOUNCED INSPECTION OF REACTOR SHUTDOWN MARGIN DETERMINATION; ISOTHERMAL TEMPERATURE COEFFICIENT, POWER COEFFICIENT OF REACTIVITY MEASUREMENT; CONTROL ROD WORTH MEASUREMENT; TARGET AXIAL FLUX DIFFERENCE; AND REACTIVITY ANOMALIES. THE INSPECTION INVOLVED 36 INSPECTOR-HOURS ONSITE BY THREE NRC INSPECTORS INCLUDING ZERO INSPECTOR-HOURS ONSITE DURING OFF-SHIFTS. NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

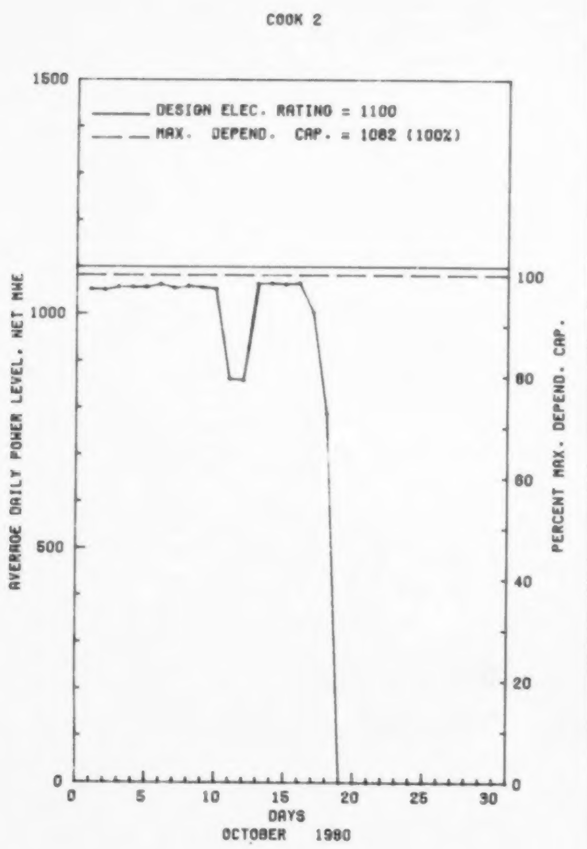
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE.			

REVIEWED BY OMPA LOEB: DATE: 12/10/80

REVIEWED BY NRR DL: DATE: 12/10/80

COOK 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-316
2. Utility Contact: W. T. Gillett (616) 465-5901
3. Licensed Thermal Power (MWt): 3391
4. Nameplate Rating (Gross MWe): 1333 x 0.85 = 1133
5. Design Electrical Rating (Net MWe): 1100
6. Maximum Dependable Capacity (Gross MWe): 1118
7. Maximum Dependable Capacity (Net MWe): 1082
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	745.0	7,320.0	24,840.0
12. Hours Reactor Was Critical	429.0	6,160.4	17,240.3
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	429.0	6,068.4	16,570.3
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,414,143	19,906,496	52,130,876
17. Gross Electrical Energy (MWH)	456,040	6,457,370	16,611,780
18. Net Electrical Energy (MWH)	439,846	6,228,971	15,996,383
19. Unit Service Factor	57.6	82.9	74.4
20. Unit Availability Factor	57.6	82.9	74.4
21. Unit Capacity Factor (MDC Net)	54.6	78.6	69.0
22. Unit Capacity Factor (DER Net)	53.7	77.4	68.4
23. Unit Forced Outage Rate	42.4	6.3	11.1

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 12/5/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
86	10/11/80	F	.0	A	5		ZZ	ZZZZZZ	REACTOR POWER REDUCED TO 55% TO REMOVE EAST MAIN FEEDPUMP TURBINE FROM SERVICE TO CHECK FEEDPUMP TURBINE CONDENSER FOR TUBE LEAKS. TWO TUBES WERE PLUGGED. REACTOR POWER RETURNED TO 100% THE SAME DAY.
87	10/12/80	F	.0	A	5		ZZ	ZZZZZZ	REACTOR POWER REDUCED TO 55% TO REMOVE EAST MAIN FEEDPUMP TURBINE FROM SERVICE TO CHECK FEEDPUMP TURBINE CONDENSER FOR TUBE LEAKS. ONE LEAKY TUBE WAS PLUGGED. REACTOR POWER RETURNED TO 100% THE SAME DAY.
88	10/17/80	F	.0	A	5		ZZ	ZZZZZZ	REACTOR POWER REDUCED TO 55% TO REMOVE EAST MAIN FEEDPUMP TURBINE FROM SERVICE TO CHECK FEEDPUMP TURBINE CONDENSER FOR TUBE LEAKS. FOUR TUBES WERE PLUGGED. REACTOR POWER RETURNED TO 100% 10/18/80.
89	10/18/80	F	316.0	A	3		HA	GENERA	UNIT TRIP. CAUSE OF TRIP WAS MAIN GENERATOR NEUTRAL GROUND RELAY ACTUATION. THE GROUND FAULT WAS LOCATED IN ONE OF THE GENERATOR STATOR BARS. REPAIRS ARE PRESENTLY IN PROGRESS. THE UNIT REMAINED OUT OF SERVICE AT THE END OF THE MONTH.

SUMMARY

COOK 2 OPERATED WITH 3 REDUCTIONS ON THE 11TH, 12TH, AND 17TH, AND ONE OUTAGE ON THE 18TH DUE TO EQUIPMENT FAILURE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

COOPER STATION

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Nebraska/Nemaha 23 miles S of Nebraska City
2.	Electric Reliability Council	Mid-Continent Area Reliability Coordination Agreement
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2381
5.	Design Electrical Rating (MWE Net)	778
6.	Date of Initial Criticality	February 21, 1974
7.	Date of Commercial Operation	July 1, 1974 (50% rating)
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Missouri River

Utility & Contractor Information

10.	Licensee	Nebraska Public Power District
11.	Corporate Address	P. O. Box 499 Columbus, Nebraska 68601
12.	Corporate Contact	J. M. Pilant, Director Licensing and Quality Assurance
13.	Architect/Engineer	Burns and Roe, Inc.
14.	Nuclear Steam Supply System	GE
15.	Constructor	Burns and Roe, Inc.

Regulatory Information

16.	IE Region responsible	IV
17.	IE Branch Chief	G. L. Madsen
18.	IE Section Chief	T. F. Westerman
19.	IE Resident Inspector	R. G. Spangler
20.	Licensing Project Manager	V. L. Rooney
21.	Docket Number	50-298
22.	License Number/Date of Issue	DPR-46, 1/18/74
23.	Public Document Room	Auburn Public Library 118 15th Street Auburn, Nebraska 68305

INSPECTION STATUS

ENFORCEMENT STATUS

TEXT OMITTED DUE TO SENSITIVITY OF DATA 9/11/80.
(7909 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

FACILITY ITEMS (PLANS AND PROCEDURES):

MANAGERIAL ITEMS:

PLANT STATUS:

OPERATING AT RATED THERMAL POWER.

LAST IE SITE INSPECTION DATE: SEPTEMBER 29 - OCTOBER 2, 1980

INSPECTION REPORT NO: 50-298/80-15

SUMMARY

INSPECTION SUMMARY

INSPECTION ON SEPTEMBER 15-18, 1980 (80-14): ROUTINE, UNANNOUNCED INSPECTION OF REVIEW OF THE SECURITY PLAN; SECURITY PROGRAM AUDIT; SECURITY ORGANIZATION; PROTECTED AREA PHYSICAL BARRIERS; VITAL AREA PHYSICAL BARRIERS; ACCESS CONTROL (IDENTIFICATION, AUTHORIZATION AND BADGING); ACCESS CONTROL (VITAL AREAS); ALARM STATIONS; DETECTION AIDS; TESTING AND MAINTENANCE; SUPPLY; RESPONSE; LOCKS, KEYS AND COMBINATIONS; FOLLOW-UP ON UNRESOLVED ITEMS. THE INSPECTION INVOLVED TWENTY-NINE (29) INSPECTOR-HOURS BY ONE (1) NRC INSPECTOR INCLUDING SIX (6) INSPECTION HOURS ACCOMPLISHED DURING OFF-SHIFT PERIODS. THE LICENSEE WAS FOUND TO BE IN COMPLIANCE WITH NRC REQUIREMENTS WITHIN THE AREAS EXAMINED DURING THE INSPECTION.

INSPECTION ON SEPTEMBER 29 - OCTOBER 2, 1980 (80-15): ROUTINE, UNANNOUNCED INSPECTION OF FOLLOWUP TO PREVIOUSLY IDENTIFIED ITEMS, AUDIT PROGRAM, AUDIT PROGRAM IMPLEMENTATION, AND FOLLOWUP ON LER'S. THIS INSPECTION INVOLVED 24 INSPECTOR HOURS ON SITE BY ONE NRC INSPECTOR. NO ITEMS OF NONCOMPLIANCE OR DEVIATION WERE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

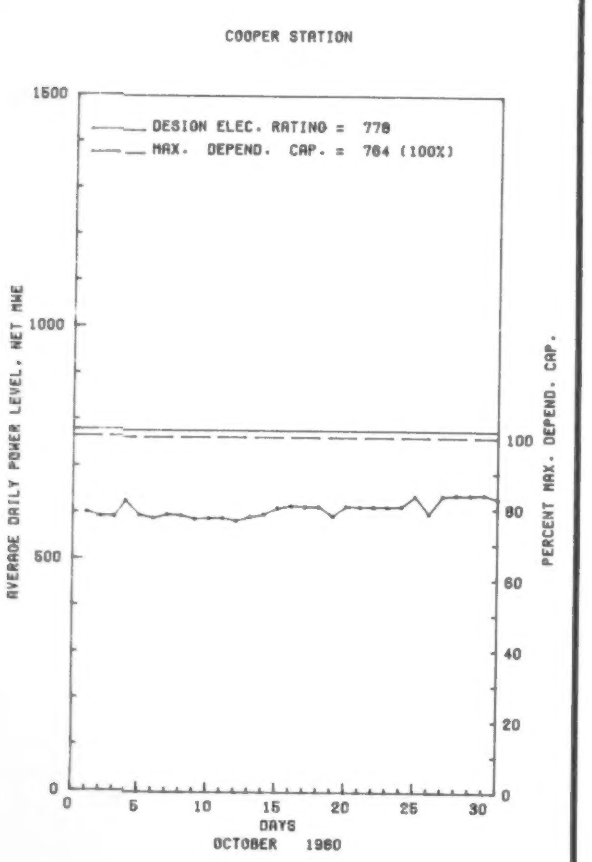
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-34 03L-0	9/8/80	10/6/80	FAILURE OF RHR-1D CIRCUIT BREAKER CHARGING SPRING TO RECYCLE
80-35 03L-0	10/1/80	10/6/80	GASKET IN OIL LINE TO THE HPCI STOP VALVE FAILED

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 12-10-80

REVIEWED BY NRR DL: V Rooney **DATE:** 12/10/80

COOPER STATION

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-298
2. Utility Contact: P. J. Borer (402) 825-3811
3. Licensed Thermal Power (MWt): 2381
4. Nameplate Rating (Gross MWe): 983 x 0.85 = 836
5. Design Electrical Rating (Net MWe): 778
6. Maximum Dependable Capacity (Gross MWe): 787
7. Maximum Dependable Capacity (Net MWe): 764
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): 640
10. Reasons for Restrictions, If Any: TEMPORARY TURBINE MODIFICATION

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	745.0	7,320.0	55,561.0
12. Hours Reactor Was Critical	745.0	5,009.7	46,247.0
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	745.0	4,915.4	45,391.0
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,677,720	10,637,472	86,820,198
17. Gross Electrical Energy (MWH)	471,867	3,110,260	27,649,108
18. Net Electrical Energy (MWH)	453,412	2,988,459	26,646,736
19. Unit Service Factor	100.0	67.2	81.7
20. Unit Availability Factor	100.0	67.2	81.7
21. Unit Capacity Factor (MDC Net)	79.7	53.4	62.8
22. Unit Capacity Factor (DER Net)	78.2	52.5	61.6
23. Unit Forced Outage Rate	[illegible]	1.8	4.3

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	NONE								

SUMMARY

COOPER STATION OPERATED UNDER A RESTRICTION FOR TURBINE MODIFICATION DURING OCTOBER WITH NO REDUCTIONS OR OUTAGES REPORTED.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

CRYSTAL RIVER 3

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Florida/Citrus 7 miles NW of Crystal River
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2452
5.	Design Electrical Rating (MWE Net)	825
6.	Date of Initial Criticality	January 14, 1977
7.	Date of Commercial Operation	March 13, 1977
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Gulf of Mexico

Utility & Contractor Information

10.	Licensee	Florida Power Corporation
11.	Corporate Address	3201 34th Street, South St. Petersburg, Florida 33733
12.	Corporate Contact	J. Hancock
13.	Architect/Engineer	Gilbert Associates, Inc.
14.	Nuclear Steam Supply System	B&W
15.	Constructor	J. A. Jones Construction Co.

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. Lewis
18.	IE Section Chief	R. Martin
19.	IE Resident Inspector	T. F. Stetka (904) 795-7677
20.	Licensing Project Manager	M. B. Fairtile
21.	Docket Number	50-302
22.	License Number/Date of Issue	DPR-72, 12/3/76
23.	Public Document Room	Crystal River Public Library 668 N.W. First Crystal River, Florida 32639

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO ENVIRONMENTAL TECHNICAL SPECIFICATION 3.1.5 THE LICENSEE DID NOT OBTAIN AND ANALYZE SAMPLES FROM TEST WELL NO. 4. CONTRARY TO 10 CFR PART 50.59(A)(1), THE LICENSEE CHANGED THE ENVIRONMENTAL TECHNICAL SPECIFICATIONS TO ELIMINATE TEST WELL NO. 4 WITHOUT PRIOR COMMISSION APPROVAL.
(8024 INFRACTION)

CONTRARY TO 10 CFR PART 50, APPENDIX B, CRITERION V, AND FSAR SECTION 1.7.1, AN INADEQUATE PROCEDURE CAUSED AN OPERATIONAL TRANSIENT THAT RESULTED IN A LOSS OF PRESSURIZER LEVEL.
(8024 INFRACTION)

OPENINGS IN VA BARRIER NOT SECURED WITH REQUIRED MATERIALS.
(8035 INFRACTION)

FAILURE TO SEARCH HAND-CARRIED PACKAGES.
(8035 INFRACTION)

FAILURE TO PROVIDE QUARTERLY EVALUATION OF SECURITY PERSONNEL.
(8035 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

- ROUTINE OPERATION.
- LAST IE SITE INSPECTION DATE: OCTOBER 4 - NOVEMBER 1, 1980
- INSPECTION REPORT NO: 50-302/80-38

SUMMARY

INSPECTION SUMMARY

- INSPECTION AUGUST 1-29 (80-28): THIS ROUTINE INSPECTION BY THE RESIDENT INSPECTORS OF PLANT OPERATIONS, SECURITY, RADIOLOGICAL CONTROLS, POST REFUELING TESTING, LICENSEE EVENTS REPORT (LERS), LICENSEE ACTION ON IE BULLETINS AND CIRCULARS, NON-ROUTINE EVENTS AND LICENSEE ACTION ON PREVIOUS INSPECTION ITEMS. FACILITY TOURS WERE CONDUCTED AND FACILITY OPERATIONS WERE OBSERVED. THE INSPECTION INVOLVED 76 HOURS ON SITE BY TWO RESIDENT INSPECTORS AND ONE REGION BASED INSPECTOR. NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION SEPTEMBER 8-12 (80-32): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 96 INSPECTOR-HOURS ON SITE. THE AREAS INSPECTED INCLUDED PRE-CRITICAL ACTIVITIES AND INITIAL CRITICALITY FOR CYCLE 3, ZERO-POWER PHYSICS TESTS, POWER ESCALATION TESTS, AND LER FOLLOWUP. OF THE FOUR AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION SEPTEMBER 14-17 (80-35): INCLUDED REVIEW OF SECURITY ORGANIZATION, VITAL AREA PHYSICAL BARRIERS; ACCESS CONTROL (IDENTIFICATION, AUTHORIZATION, BADGING AND SEARCH); TESTING AND MAINTENANCE; LIGHTING AND FOLLOWUP ON PREVIOUS ENFORCEMENT MATTERS. THE INSPECTION INVOLVED 28 INSPECTOR-HOURS ON SITE BY ONE NRC INSPECTOR. THE INSPECTION WAS BEGUN DURING AN OFFSHIFT PERIOD; 4 INSPECTION HOURS WERE ACCOMPLISHED DURING OFFSHIFT PERIODS. OF THE SEVEN AREAS EXAMINED DURING THE INSPECTION, THREE APPARENT ITEMS OF NONCOMPLIANCE WERE IDENTIFIED IN THREE AREAS. (1) PHYSICAL BARRIERS - TWO SECTIONS OF VITAL AREA BARRIER NOT CONSTRUCTED OF REQUIRED MATERIAL; (2) ACCESS CONTROL - FAILURE TO SEARCH HAND-CARRIED PACKAGES ENTERING THE PROTECTED AREA; (3) TESTING AND MAINTENANCE - FAILURE TO PROVIDE DOCUMENTED EVIDENCE OF QUARTERLY PERSONNEL EVALUATION OF EQUIPMENT USE PERFORMANCE.

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-031/03L-0	08/06/80	08/29/80	EIGHT AREAS TO BE ENVIRONMENTALLY MONITORED WERE MISSED.
80-034/03L-0	08/20/80	09/02/80	MAXIMUM CIRCULATING WATER DISCHARGE TEMPERATURE WAS EXCEEDED.

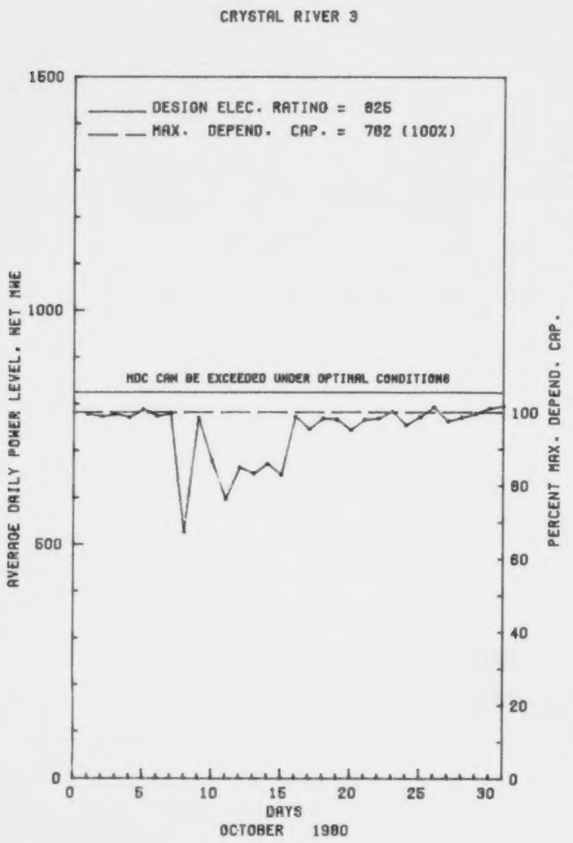
REVIEWED BY OMPA LOEB: ______ **DATE:** 12/10/80

REVIEWED BY NRR DL: ______ **DATE:** 12/10/80

best copy available

CRYSTAL RIVER 3

AVERAGE POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-302
2. Utility Contact: M.W. CULVER (904) 795-6486
3. Licensed Thermal Power (MWt): 2452
4. Nameplate Rating (Gross MWe): 989 x 0.9 = 890
5. Design Electrical Rating (Net MWe): 825
6. Maximum Dependable Capacity (Gross MWe): 821
7. Maximum Dependable Capacity (Net MWe): 782
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	745.0	7,320.0	31,896.0
12. Hours Reactor Was Critical	740.7	3,285.2	18,435.4
13. Reactor Reserve Shutdown Hours	4.2	12.3	1,129.2
14. Hours Generator On-Line	739.2	3,220.6	17,920.9
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,716,479	7,055,644	39,372,555
17. Gross Electrical Energy (MWH)	580,161	2,395,900	13,367,668
18. Net Electrical Energy (MWH)	551,436	2,273,428	12,665,068
19. Unit Service Factor	99.2	44.0	56.2
20. Unit Availability Factor	99.2	44.0	56.2
21. Unit Capacity Factor (MDC Net)	94.7	39.7	50.8
22. Unit Capacity Factor (DER Net)	89.7	37.6	48.1
23. Unit Forced Outage Rate	.8	9.2	29.7

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-17	10/08/80	F	5.8	A	3		HA	INSTRU	DURING SYSTEM SWITCHING, VOLTAGE REGULATOR OSCILLATION RESULTED IN AN OVEREXCITATION TRIP OF THE TURBINE WHEN THE REGULATOR WAS PLACED IN "TEST" POSITION. THE TRIPPING OF THE TURBINE AUTOMATICALLY TRIPPED THE REACTOR.
80-18	10/09/80	S	13.1	H	5		HF	MOTORX	REDUCED POWER TO 75% FP TO MODIFY THE COOLING WATER PIPING FOR CWP-1D MOTOR.
80-19	10/10/80	S	16.4	H	5		HF	MOTORX	REDUCED POWER TO 75% FP TO MODIFY THE COOLING WATER PIPING FOR CWP-1A MOTOR.
80-20	10/11/80	S	11.2	H	5		HF	MOTORX	REDUCED POWER TO 75% FP TO MODIFY THE COOLING WATER PIPING FOR CWP-1B MOTOR.
80-21	10/12/80	S	11.3	H	5		HF	MOTORX	REDUCED POWER TO 75% FP TO MODIFY THE COOLING WATER PIPING FOR CWP-1C MOTOR.
80-22	10/13/80	F	3.4	A	5		HA	VALVEX	VIBRATION PROBLEMS ON GOVERNOR VALVE REQUIRED REDUCTION OF POWER TO APPROXIMATELY 85% FP TO CLOSE THE VALVE.
80-23	10/13/80	F	383.7	A	5		HA	VALVEX	PLANT MAXIMUM POWER HELD TO 95 - 97% FP BECAUSE OF GOVERNOR VALVE VIBRATION UNTIL VALVE REPROGRAMMING ON 10/29.
80-24	10/14/80	F	33.7	A	5		HC	HTEXCH	REDUCED POWER TO FIND A SALTWATER LEAK IN "D" CONDENSER WATERBOX. LEAK FOUND AND PLUGGED.
80-25	10/16/80	S	7.9	B	5		HA	VALVEX	REDUCED POWER TO ATTEMPT REPROGRAMMING OF GOVERNOR VALVE.

SUMMARY

CRYSTAL RIVER 3 OPERATED NEAR FULL POWER UNTIL THE 13TH OF THE MONTH WITH THE EXCEPTION OF ONE OUTAGE AND FOUR POWER REDUCTIONS. FROM OCTOBER 13TH TO OCTOBER 29TH, POWER WAS LIMITED BY A FAULTY GOVERNOR VALVE AND THERE WAS AN ADDITIONAL POWER REDUCTION UNRELATED TO THE GOVERNOR VALVE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

DAVIS-BESSE 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Ohio/Ottawa 21 miles E of Toledo
2.	Electric Reliability Council	East Central Area Reliability Coordination Agreement
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2772
5.	Design Electrical Rating (MWE Net)	906
6.	Date of Initial Criticality	August 12, 1977
7.	Date of Commercial Operation	11/21/77 (25%); 12/19/77 (40%) 01/23/78 (75%); 07/31/78 (100%)
8.	Condenser Cooling Method	Cooling Tower
9.	Condenser Cooling Water Source	Lake Erie

Utility & Contractor Information

10.	Licensee	Toledo Edison Co.
11.	Corporate Address	Edison Plaza, 300 Madison Avenue Toledo, Ohio 43652
12.	Corporate Contact	R. P. Crouse, V. P., Nuclear (419)259-5221
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	B&W
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	R. F. Warnick
19.	IE Resident Inspector	L. Reyes (419) 898-2775
20.	Licensing Project Manager	D. Garner
21.	Docket Number	50-346
22.	License Number/Date of Issue	NPF-3, 4/22/77
23.	Public Document Room	Ida Rupp Public Library 310 Madison Street Port Clinton, OH 43452

INSPECTION STATUS

ENFORCEMENT STATUS

TECHNICAL SPECIFICATION 6.8.1 REQUIRES THAT WRITTEN PROCEDURES RECOMMENDED IN APPENDIX A OF REGULATORY GUIDE 1.33, NOVEMBER 1972, SHALL BE IMPLEMENTED. APPENDIX A OF REGULATORY GUIDE 1.33, NOVEMBER 1972, INCLUDES ADMINISTRATIVE PROCEDURES. ADMINISTRATIVE PROCEDURE AD 1839.00, REVISION 6, "STATION OPERATIONS" REQUIRES THAT THE OPERATIONS ENGINEER SHALL MAKE PERIODIC DETAILED TOURS OF THE UNIT CONDUCTED AT A MINIMUM OF ONCE PER WEEK. EACH OF THESE TOURS SHALL BE DOCUMENTED BY AN ENTRY IN THE UNIT LOG. CONTRARY TO THE ABOVE, NO ENTRIES OF PLANT TOURS BY THE OPERATIONS ENGINEER WERE MADE FOR THE PERIOD OF JUNE 11 THROUGH JUNE 25, 1980.
(8023 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENTS

+ ON SEPTEMBER 5, 1980 WHILE REPLACING THE OIL FOR THE EMERGENCY DIESEL GENERATOR (EDG) NO. 1, A SECTION OF A 5/8 INCH BOLT AND SOME METAL SHAVINGS WERE FOUND IN THE ENGINE OIL SUMP. FURTHER INVESTIGATION SHOWS THAT THE BOLT WAS PART OF THE TURBOCHARGER SPRING DRIVE GEAR ASSEMBLY. THE THREE REMAINING BOLTS IN THE ASSEMBLY WERE LOOSE AND EXHIBITED SEVERE THREAD DAMAGE. THE EDG IS A GM-EMD 20 CYLINDER MODEL 20-645-E4. A BULLETIN IS BEING PREPARED IN REGION III. (80-25)

PLANT STATUS

+ THE REFUELING/MAINTENANCE OUTAGE CONTINUED DURING THE MONTH, AND WAS CONCLUDED ON NOVEMBER 1, 1980.

+LAST IE SITE INSPECTION DATE: SEPTEMBER 9-12, 1980

+ INSPECTION REPORT NO: 80-26

SUMMARY

INSPECTION SUMMARY

+ INSPECTION ON AUGUST 18-22, 25-29; SEPTEMBER 2-5, 8-12 AND 15-19, (80-25): ROUTINE RESIDENT INSPECTION OF FOLLOWUP ACTION ON PREVIOUS INSPECTION FINDINGS, MONTHLY MAINTENANCE OBSERVATION, MONTHLY SURVEILLANCE OBSERVATION, IE BULLETIN FOLLOWUP, IE CIRCULAR FOLLOWUP, FOLLOWUP ON LICENSEE EVENT REPORTS, INSPECTION DURING LONG-TERM SHUTDOWN, AND ORGANIZATION AND ADMINISTRATION. THE INSPECTION INVOLVED 226 INSPECTOR-HOURS ONSITE BY FOUR NRC INSPECTORS INCLUDING 48 INSPECTOR-HOURS ONSITE DURING OFF-SHIFTS. OF THE EIGHT AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN SEVEN AREAS; ONE APPARENT ITEM OF NONCOMPLIANCE WAS IDENTIFIED IN THE OTHER AREA (FAILURE TO USE THE LATEST REVISION OF AN APPROVED PROCEDURE).

+ INSPECTION ON SEPTEMBER 9-12, (80-26): INCLUDED A REVIEW OF THE PROTECTED AREA PHYSICAL BARRIERS; VITAL AREA PHYSICAL BARRIERS; ACCESS CONTROL (VITAL AREAS); ACCESS CONTROL (SEARCH) AND LICENSEE ACTION TAKEN ON PREVIOUS INSPECTION/INVESTIGATION FINDINGS. THE INSPECTION INVOLVED 54 INSPECTOR-HOURS ONSITE BY TWO NRC INSPECTORS. THE INSPECTION BEGAN DURING AN OFFSHIFT PERIOD; 16 INSPECTOR-HOURS WERE ACCOMPLISHED DURING OFFSHIFT PERIODS. THE LICENSEE WAS FOUND TO BE IN COMPLIANCE WITH NRC REQUIREMENTS WITHIN THE AREAS EXAMINED DURING THIS INSPECTION EXCEPT FOR THE FOLLOWING TWO ITEMS. ACCESS CONTROL (VITAL AREA) - TWO VITAL AREA PORTALS WERE FOUND UNSECURED AND UNATTENDED. THIS IS A REPEAT ITEM OF NONCOMPLIANCE FROM A PREVIOUS SECURITY INSPECTION; AND ACCESS CONTROL (SEARCH) - SEARCHES WERE NOT PERFORMED OF SEVERAL HAND-CARRIED PACKAGES. TWO PREVIOUS ITEMS OF NONCOMPLIANCE, WHICH WERE IDENTIFIED IN IE INSPECTION REPORTS 50-346/80-02 AND 50-346/80-13, REMAIN OPEN. ALL REMAINING ITEMS WERE CLOSED.

REPORTS RECEIVED FROM LICENSEE

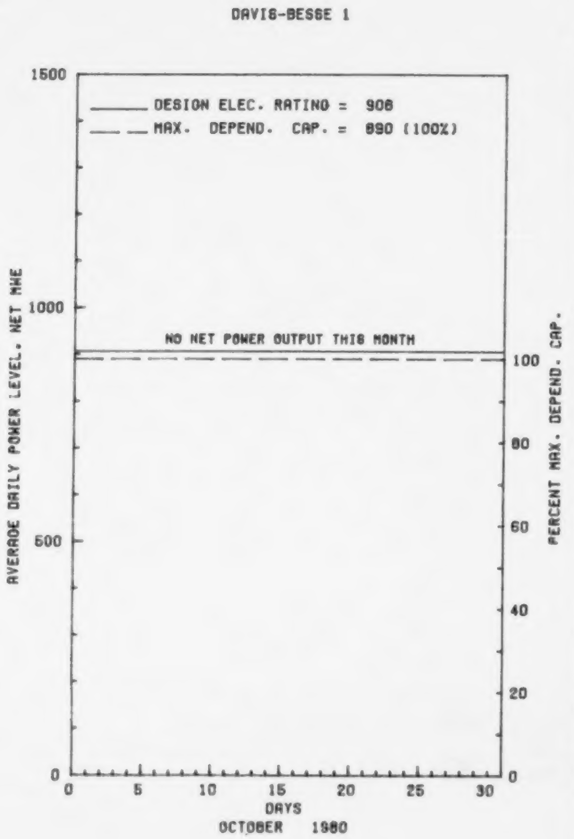
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-54/ 03L-0	06/27/80	07/23/80	SOURCE RANGE NUCLEAR INSTRUMENTATION CHANNEL 2 FAILURE.
80-62/ 03L-0	09/05/80	09/18/80	PROTECTIVE FEEDWATER FLOODING CURBS NOT INSTALLED PER FSAR.
80-63/ 03L-0	08/23/80	09/18/80	DAMPER INOPERABLE.
SUPPLEMENT - 10/07/80			
80-64/ 03L-0	08/22/80	09/19/80	LOSS OF ESSENTIAL INSTRUMENT PANEL.
80-65/ 03L-0	08/26/80	09/23/80	LOSS OF CONTROL POWER ON A 4160 VAC ESSENTIAL BUS.
80-68/ 03L-0	09/01/80	09/29/80	INDICATIONS FOR METEOROLOGICAL INSTRUMENTATION WERE LOST.
80-69/ 03L-0	09/02/80	10/02/80	EMERGENCY DG IDLER GEAR BOLT FAILURE.
80-70/ 03L-0	08/28/80	10/03/80	DIESEL FIRE PUMP WOULD NOT START AT REQUIRED PRESSURE.

REVIEWED BY OMPA LOEB: R Boba **DATE:** 12/10/80

REVIEWED BY NRR DL: Daniel J Garner **DATE:** 12/11/80

DAVIS-BESSE 1

AVERAGE POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-346
2. Utility Contact: BILAL SARSOUR (419) 259-5000 X236
3. Licensed Thermal Power (MWt): 2772
4. Nameplate Rating (Gross MWe): 1069 x 0.9 = 962
5. Design Electrical Rating (Net MWe): 906
6. Maximum Dependable Capacity (Gross MWe): 934
7. Maximum Dependable Capacity (Net MWe): 890
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	745.0	7,320.0	27,845.0
12. Hours Reactor Was Critical	.0	2,078.2	13,042.4
13. Reactor Reserve Shutdown Hours	.0	.0	2,875.8
14. Hours Generator On-Line	.0	2,008.7	11,883.4
15. Unit Reserve Shutdown Hours	.0	.0	1,728.2
16. Gross Thermal Energy (MWH)	0	4,687,304	24,886,811
17. Gross Electrical Energy (MWH)	0	1,583,559	8,307,070
18. Net Electrical Energy (MWH)	0	1,483,787	7,654,365
19. Unit Service Factor	.0	27.4	43.2
20. Unit Availability Factor	.0	27.4	49.0
21. Unit Capacity Factor (MDC Net)	.0	22.8	32.0
22. Unit Capacity Factor (DER Net)	.0	22.4	32.3
23. Unit Forced Outage Rate	.0	14.3	25.6

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 11/06/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
4	04/07/80	S	745.0	C	4				SCHEDULED MAINTENANCE AND REFUELING CONTINUES.

SUMMARY

DAVIS-BESSE REMAINED OFF-LINE FOR THE MONTH OF OCTOBER AS REFUELING AND MAINTCHANCE CONTINUED.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

DRESDEN 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Illinois/Grundy 9 miles E of Morris
2.	Electric Reliability Council	Mid-America Interpool Network
3.	Type of Reactor	BWR
4.	Power Level (MWT)	700
5.	Design Electrical Rating (MWE Net)	200
6.	Date of Initial Criticality	October 15, 1959
7.	Date of Commercial Operation	July 4, 1960
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Kankakee River

Utility & Contractor Information

10.	Licensee	Commonwealth Edison Co.
11.	Corporate Address	P. O. Box 767 Chicago, Illinois 60690
12.	Corporate Contact	C. Reed
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	GE
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	R. Spessard
19.	IE Resident Inspector	T. Tongue (815) 942-9267
20.	Licensing Project Manager	Paul W. O'Connor
21.	Docket Number	50-010
22.	License Number/Date of Issue	DPR-2 issued 9/28/59
23.	Public Document Room	Morris Public Library 604 Liberty Street Morris, Illinois 60451

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

UNIT 1 SHUTDOWN ON OCTOBER 31, 1978 FOR AN EXTENDED OUTAGE TO UPGRADE ECCS, CHEMICAL CLEANING AND REFUELING. OUTAGE EXPECTED TO LAST THROUGH MID-1986.

*LAST IE SITE INSPECTION DATE: SEPTEMBER 22-26, 1980

* INSPECTION REPORT NO: 80-16

SUMMARY

INSPECTION SUMMARY

* INSPECTION ON AUGUST 3 THROUGH SEPTEMBER 6, (80-14): ROUTINE, UNANNOUNCED RESIDENT INSPECTION OF MONTHLY SURVEILLANCE OBSERVATION; MONTHLY MAINTENANCE OBSERVATION; INSPECTION DURING LONG-TERM SHUT DOWN; IE BULLETIN FOLLOWUP; IE CIRCULAR FOLLOWUP; AND FOLLOWUP ON A PLANT INCIDENT. THE INSPECTION INVOLVED 283 INSPECTOR-HOURS ONSITE BY FOUR NRC INSPECTORS INCLUDING 49.5 HOURS ONSITE DURING OFF-SHIFTS. OF THE SEVEN AREAS INSPECTED, THERE WERE NO ITEMS OF NONCOMPLIANCE IDENTIFIED.

* INSPECTION ON SEPTEMBER 15-17, (80-15): SPECIAL, UNANNOUNCED INSPECTION OF CIRCUMSTANCES PERTAINING TO ALLEGATIONS MADE BY UNIDENTIFIED INDIVIDUALS TO A CHICAGO NEWSPAPER THAT 300,000 TO 400,000 GALLONS OF WATER CONTAMINATED WITH LOW LEVELS OF RADIOISOTOPES HAD BEEN RELEASED TO THE ENVIRONMENT OVER A PERIOD OF ONE TO TWO MONTHS PRIOR TO SEPTEMBER 5, 1980; INCLUDING A REVIEW OF METHODS OF ACCOUNTING FOR INVENTORIES OF CONTAMINATED WATER; SPECIAL AND ROUTINE ENVIRONMENTAL SAMPLE RESULTS; INDEPENDENT INSPECTION OF POTENTIAL RELEASE FLOW PATHS; PROGRAMS TO DETECT LEAKAGE FROM THE STATION; AND, THE COLLECTION AND ANALYSIS OF INDEPENDENT ENVIRONMENTAL SAMPLES. THE INSPECTION INVOLVED 79 INSPECTOR-HOURS ONSITE BY NINE NRC INSPECTORS INCLUDING 11.5 INSPECTOR-HOURS ONSITE DURING OFF-SHIFTS. NO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED. ALTHOUGH IT COULD NOT BE DETERMINED WHETHER A LARGE VOLUME OF WATER DID OR DID NOT ESCAPE FROM THE STATION, THERE WAS NO DETECTABLE THREAT TO THE HEALTH AND SAFETY OF THE PUBLIC.

* INSPECTION ON SEPTEMBER 22-26, (80-16): INCLUDED A REVIEW OF SECURITY PLAN; SECURITY PROGRAM AUDIT; SECURITY ORGANIZATION; PROTECTED AREA PHYSICAL BARRIERS; VITAL AREA PHYSICAL BARRIERS; LIGHTING; ACCESS CONTROL (IDENTIFICATION, AUTHORIZATION, AND BADGING); ACCESS CONTROL (SEARCH); ACCESS CONTROL (ESCORTING); ACCESS CONTROL (VITAL AREAS); ALARM STATIONS; DETECTION AIDS; ASSESSMENT AIDS; COMMUNICATIONS; TESTING AND MAINTENANCE; COMPENSATORY MEASURES; POWER SUPPLY; RESPONSE; LOCKS, KEYS, AND COMBINATIONS; RECORDS AND REPORTS; FOLLOWUP ON PREVIOUS ITEMS OF NONCOMPLIANCE/DEVIATIONS. THE INSPECTION INVOLVED 84 INSPECTOR-HOURS ONSITE BY TWO NRC INSPECTORS. THE INSPECTION BEGAN DURING DAY SHIFT PERIOD; 26 INSPECTOR HOURS WERE ACCOMPLISHED DURING OFF-SHIFT PERIODS. THE LICENSEE WAS FOUND TO BE IN COMPLIANCE WITH NRC REQUIREMENTS WITHIN THE ABOVE AREAS EXAMINED DURING THIS INSPECTION EXCEPT FOR THE FOLLOWING ONE ITEM. ACCESS CONTROL (IDENTIFICATION, AUTHORIZATION, AND BADGING) - LICENSEE RECORDS FAILED TO CONTAIN PROPER SCREENING DOCUMENTATION ON CERTAIN CONTRACTOR PERSONNEL.

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE			

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 12-10-80

REVIEWED BY NRR DL: Paul W O'Connor **DATE:** 12/11/80

DRESDEN 1

AVERAGE DAILY POWER LEVEL (MWe)

OPERATING STATUS

1. Docket Number: 50-010
2. Utility Data Prepared By. S. A. CIESZKIEWICZ (815) 942-2920 X489
3. Licensed Thermal Power (MWt): 700
4. Nameplate Rating (Gross MWe): 245.5 x 0.85 = 209
5. Design Electrical Rating (Net MWe): 200
6. Maximum Dependable Capacity (Gross MWe): 0
7. Maximum Dependable Capacity (Net MWe): 0
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons

None

9. Power Level To Which Restricted, If Any (Net MWe): None
10. Reasons For Restrictions, If Any:

SHUTDOWN INDEFINITELY

	This Month	Yr	ulative
11. Hours In Reporting Period			
12. Number Of Hours Reactor Was Critical			
13. Reactor Reserve Shutdown Hour			
14. Hours Generator On-			
15. Unit Reserve Shutd			
16. Gross Thermal Energy Generated (MWH)			
17. Gross Electrical Energy Generated (MWH)			
18. Net Electrical Energy Generated (MWH)			
19. Unit Service Factor			
20. Unit Availability Factor			
21. Unit Capacity Factor (Using MDC Net) 1/			
22. Unit Capacity Factor (Using DER Net) 1/			
23. Unit Forced Outage Rate			

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

None

25. If Shut Down At End Of Report Period, Estimated Date of Startup: 06/86

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type^1	Duration (Hours)	Reason^2	Method of Shutting Down Reactor^3	Licensee Event Report No.	System Code^4	Component Code^5	Cause & Corrective Action to Prevent Recurrence
SEE SUMMARY BELOW.									

SUMMARY

DRESDEN 1 IS SHUTDOWN UNTIL APPROXIMATELY JUNE, 1986 FOR THE FOLLOWING REASONS

1) HPCI INSTALLATION FOR ECCS UPGRADING
2) EQUIPMENT UPGRADE TO IEEE 279
3) MAJOR INSERVICE INSPECTION
4) CHEMICAL CLEANING
5) REFUEL

LEGEND

^{1}Type:
F--Forced
S--Scheduled

^{2}Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

^{3}Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

^{4}Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

^{5}Exhibit H:
Same source.

DRESDEN 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Illinois/Grundy 9 miles E of Morris
2.	Electric Reliability Council	Mid-America Interpool Network
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2527
5.	Design Electrical Rating (MWE Net)	794
6.	Date of Initial Criticality	January 7, 1970
7.	Date of Commercial Operation	JULY, 1970
8.	Condenser Cooling Method	Cooling Lake
9.	Condenser Cooling Water Source	Kankakee and Des Plaines Rivers

Utility & Contractor Information

10.	Licensee	Commonwealth Edison Co.
11.	Corporate Address	P. O. Box 767 Chicago, Illinois 60690
12.	Corporate Contact	C. Reed
13.	Architect/Engineer	Sargent & Lundy
14.	Nuclear Steam Supply System	GE
15.	Constructor	United Engineers & Constructors, Inc.

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	R. Spessard
19.	IE Resident Inspector	T. Tongue (815) 942-0267
20.	Licensing Project Manager	Paul W. O'Connor
21.	Docket Number	50-237
22.	License Number/Date of Issue	DPR-19 issued 12/22/69
23.	Public Document Room	Morris Public Library 604 Liberty Street Morris, Illinois 60451

INSPECTION STATUS

ENFORCEMENT STATUS

10 CFR 20.105 PROHIBITS POSSESSION, USE, OR TRANSFER OF LICENSED MATERIAL IN A MANNER THAT WOULD CREATE IN AN UNRESTRICTED AREA THE POTENTIAL FOR AN INDIVIDUAL TO RECEIVE A DOSE GREATER THAN TWO MILLIREMS IN ONE HOUR OR GREATER THAN 100 MILLIREMS IN SEVEN CONSECUTIVE DAYS. CONTRARY TO THE ABOVE, THE LICENSEE FOUND IN THE SCRAPYARD, AN UNRESTRICTED AREA, MATERIAL THAT PRODUCED AN EXPOSURE RATE OF APPROXIMATELY 10 MR PER HOUR AT A DISTANCE OF THREE FEET.
(8013 INFRACTION)

10 CFR 20.203(C)(2) LISTS THREE ALTERNATIVE REQUIREMENTS FOR CONTROL OF ACCESS TO HIGH RADIATION AREAS. CONTRARY TO THESE REQUIREMENTS, ON JUNE 21, 1980, THE BARRICADE OF THE ENTRY TO A HIGH RADIATION AREA AT THE UNIT 2 WEST CRD ACCUMULATOR BANK WAS NOT SECURED, LOCKED, EQUIPPED WITH AN AUDIBLE ALARM SYSTEM, OR EQUIPPED WITH A CONTROL DEVICE TO REDUCE THE RADIATION LEVEL.
(8013 INFRACTION)

TECHNICAL SPECIFICATION 6.2.B REQUIRES THAT RADIATION CONTROL PROCEDURES BE MAINTAINED, MADE AVAILABLE TO ALL PERSONNEL, AND ADHERED TO. THE DOCUMENT "DRESDEN NUCLEAR STATION RADIATION CONTROL STANDARDS" IS TO PROCEDURE THAT ESTABLISHED CONTROL OVER PERSONAL RADIATION EXPOSURES. THE FOLLOWING EXAMPLES OF FAILURE TO FOLLOW THIS PROCEDURE WERE IDENTIFIED DURING THE APPRAISAL. CONTRARY TO THE ABOVE REQUIREMENTS: (A) LAUNDERED PROTECTIVE CLOTHING IN THE CLEAN LAUNDRY STORAGE BINS IN THE UNIT 1 TRACK WAY WAS FOUND READING TO 22,000 COUNTS PER MINUTE (CPM) IN CONTRAST TO THE "STANDARDS" LIMIT OF 3,000 CPM; AND (B) MATERIAL READING 10 MR PER HOUR AT THREE FEET WAS FOUND IN THE SCRAP YARD IN CONTRAST TO THE "STANDARDS" UNCONDITIONAL RELEASE LIMIT OF 100 CPM.
(8013 INFRACTION)

TECHNICAL SPECIFICATION 6.2.B STATES, "RADIATION CONTROL PROCEDURES SHALL BE MAINTAINED, MADE AVAILABLE TO ALL STATION PERSONNEL AND ADHERED TO. THESE PROCEDURES SHALL SHOW PERMISSIBLE RADIATION EXPOSURE AND SHALL BE CONSISTENT WITH THE REQUIREMENTS OF 10 CFR 20. THIS RADIATION PROTECTION PROGRAM SHALL BE ORGANIZED TO MEET THE REQUIREMENTS OF 10 CFR 20." THE DRESDEN RADIATION PROTECTION MANUAL STATES IN PART, "WHEN RADIATION WORK INVOLVED RAISING RADIOACTIVE MATERIALS IN THE POOLS ABOVE ESTABLISHED RADIATION LEVELS, UNCOVERING CONTAMINATED MATERIALS, VALVING OR OPENING PROCESS LINES, OR DISASSEMBLING RADIOACTIVE EQUIPMENT, RADIATION PROTECTION SHALL BE INFORMED BEFORE THE FACT SO THAT A PROPER EVALUATION OF THE RADIOLOGICAL CONSEQUENCES MAY BE MADE." CONTRARY TO THE ABOVE, ON AUGUST 12, 1980, THE RESIDENT INSPECTORS OBSERVED SEVERAL LICENSEE PERSONNEL WORKING ON A CONTAMINATED CRD PUMP WITHOUT WEARING ADEQUATE PROTECTIVE CLOTHING AND WITHOUT THE WORK AREA BEING PROPERLY POSTED. ADDITIONALLY, THE INSPECTORS DETERMINED THAT RADIATION PROTECTION HAD NOT BEEN NOTIFIED TO PERFORM AN EVALUATION OF THE RADIOLOGICAL CONSEQUENCES ASSOCIATED WITH THIS JOB. FURTHER REVIEW BY THE INSPECTORS DISCLOSED THAT THE DRESDEN OUTAGE OR WORK REQUEST FORMS DO NOT CONTAIN A REQUIREMENT FOR WORKERS TO OBTAIN RADIATION PROTECTION CONCURRENCE, WHEN REQUIRED, FOR A SPECIFIC JOB.
(8017 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

UNIT OPERATING ROUTINELY IN COASTDOWN MODE.

*LAST IE SITE INSPECTION DATE: SEPTEMBER 22-26, 1980

* INSPECTION REPORT NO: 80-19

SUMMARY

INSPECTION SUMMARY

* INSPECTION ON AUGUST 3 THROUGH SEPTEMBER 6, (80-17): ROUTINE, UNANNOUNCED RESIDENT INSPECTION OF OPERATIONAL SAFETY VERIFICATION; MONTHLY SURVEILLANCE OBSERVATION; MONTHLY MAINTENANCE OBSERVATION; IE BULLETIN FOLLOWUP; IE CIRCULAR FOLLOWUP; AND FOLLOWUP ON A PLANT INCIDENT. THE INSPECTION INVOLVED 283 INSPECTOR-HOURS ONSITE BY FOUR NRC INSPECTORS INCLUDING 49.5 HOURS ONSITE DURING OFF-SHIFTS. OF THE SEVEN AREAS INSPECTED, THERE WERE NO ITEMS OF NONCOMPLIANCE IDENTIFIED IN SIX AREAS. THERE WAS ONE ITEM OF NONCOMPLIANCE (LICENSEE PERSONNEL FAILED TO FOLLOW RADIATION PROTECTION PROCEDURES) IDENTIFIED IN ONE AREA.

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-31/ 03L-0	08/27/80	09/19/80	UNIT 1 DIESEL DRIVEN FIRE PUMP WAS OUT OF SERVICE FOR GREATER THAN 7 DAYS, CONTRARY TO TECHNICAL SPECIFICATIONS.
80-32/ 03L-0	08/29/80	09/19/80	DIESEL OIL FOR FIRE PUMP WAS NOT SAMPLED MONTHLY AS REQUIRED BY TECHNICAL SPECIFICATIONS.
80-33/ 03L-0	09/04/80	09/29/80	LEAKING LPCI HEAT EXCHANGER TUBES, DETECTED BY ABNORMAL TORUS MAKEUP REQUIREMENTS.
80-34/ 03L-0	09/08/80	09/19/80	DURING A QA AUDIT IT WAS DETERMINED THAT THE RECORDS FOR THE 1979 FIRE PROTECTION FLUSH, REQUIRED BY T.S. WERE LOST.
80-36/ 03L-0	09/11/80	10/02/80	QA AUDIT DISCLOSED THAT CARDOX SYSTEM HEAT DETECTOR SURVEILLANCE WAS NOT COMPLETED WITHIN ALLOWABLE T.S. FREQUENCY.
80-37/ 03L-0	09/15/80	10/07/80	CONTROL ROD G-8 BECAME UNCOUPLED PROBABLY BECAUSE OF A DETACHED INTERNAL FILTER.

* INSPECTION ON SEPTEMBER 15-17, (80-18): SPECIAL, UNANNOUNCED INSPECTION OF CIRCUMSTANCES PERTAINING TO ALLEGATIONS MADE BY UNIDENTIFIED INDIVIDUALS TO A CHICAGO NEWSPAPER THAT 300,000 TO 400,000 GALLONS OF WATER CONTAMINATED WITH LOW LEVELS OF RADIOISOTOPES HAD BEEN RELEASED TO THE ENVIRONMENT OVER A PERIOD OF ONE TO TWO MONTHS PRIOR TO SEPTEMBER 5, 1980; INCLUDING A REVIEW OF METHODS OF ACCOUNTING FOR INVENTORIES OF CONTAMINATED WATER; SPECIAL AND ROUTINE ENVIRONMENTAL SAMPLE RESULTS; INDEPENDENT INSPECTION OF POTENTIAL RELEASE FLOW PATHS; PROGRAMS TO DETECT LEAKAGE FROM THE STATION; AND, THE COLLECTION AND ANALYSIS OF INDEPENDENT ENVIRONMENTAL SAMPLES. THE INSPECTION INVOLVED 79 INSPECTOR-HOURS ONSITE BY NINE NRC INSPECTORS INCLUDING 11.5 INSPECTOR-HOURS ONSITE DURING OFF-SHIFTS. NO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED. ALTHOUGH IT COULD NOT BE DETERMINED WHETHER A LARGE VOLUME OF WATER DID OR DID NOT ESCAPE FROM THE STATION, THERE WAS NO DETECTABLE THREAT TO THE HEALTH AND SAFETY OF THE PUBLIC.

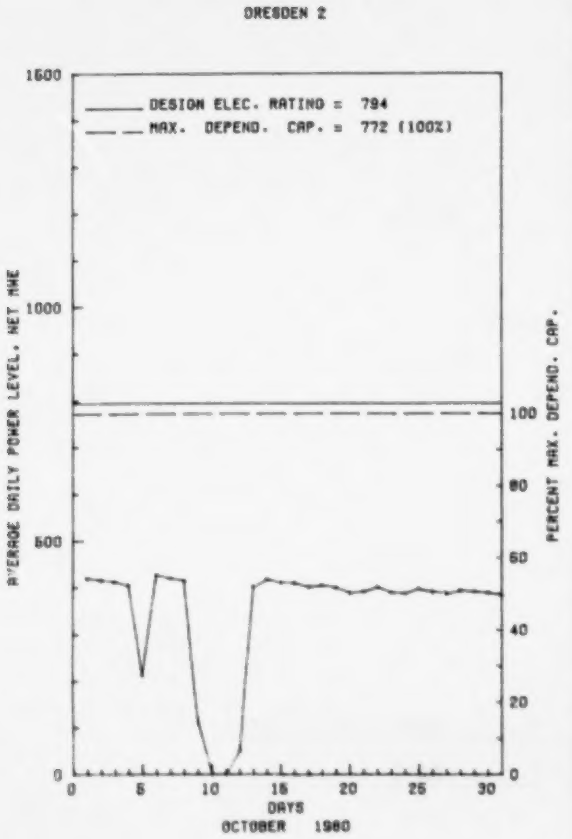
* INSPECTION ON SEPTEMBER 22-26, (80-19): INCLUDED A REVIEW OF SECURITY PLAN; SECURITY PROGRAM AUDIT; SECURITY ORGANIZATION; PROTECTED AREA PHYSICAL BARRIERS; VITAL AREA PHYSICAL BARRIERS; LIGHTING; ACCESS CONTROL (IDENTIFICATION, AUTHORIZATION, AND BADGING); ACCESS CONTROL (SEARCH); ACCESS CONTROL (ESCORTING); ACCESS CONTROL (VITAL AREAS); ALARM STATIONS; DETECTION AIDS; ASSESSMENT AIDS; COMMUNICATIONS; TESTING AND MAINTENANCE; COMPENSATORY MEASURES; POWER SUPPLY; RESPONSE; LOCKS, KEYS, AND COMBINATIONS; RECORDS AND REPORTS; FOLLOWUP ON PREVIOUS ITEMS OF NONCOMPLIANCE/DEVIATIONS. THE INSPECTION INVOLVED 84 INSPECTOR-HOURS ONSITE BY TWO NRC INSPECTORS. THE INSPECTION BEGAN DURING DAY SHIFT PERIOD; 26 INSPECTOR HOURS WERE ACCOMPLISHED DURING OFF-SHIFT PERIODS. THE LICENSEE WAS FOUND TO BE IN COMPLIANCE WITH NRC REQUIREMENTS WITHIN THE ABOVE AREAS EXAMINED DURING THIS INSPECTION EXCEPT FOR THE FOLLOWING ONE ITEM. ACCESS CONTROL (IDENTIFICATION, AUTHORIZATION, AND BADGING) - LICENSEE RECORDS FAILED TO CONTAIN PROPER SCREENING DOCUMENTATION ON CERTAIN CONTRACTOR PERSONNEL.

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 12-10-80

REVIEWED BY NRR DL: Paul W. O'Connor **DATE:** 12.11.80

DRESDEN 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-237
2. Utility Contact: S. A. CIESZKIEWICZ (815) 942-2920 X489
3. Licensed Thermal Power (MWt): 2527
4. Nameplate Rating (Gross MWe): 920 x 0.9 = 828
5. Design Electrical Rating (Net MWe): 794
6. Maximum Dependable Capacity (Gross MWe): 812
7. Maximum Dependable Capacity (Net MWe): 772
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	745.0	7,320.0	91,776.0
12. Hours Reactor Was Critical	678.2	7,049.9	71,786.6
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	653.2	6,896.6	68,159.1
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	923,773	14,065,886	135,707,718
17. Gross Electrical Energy (MWH)	278,638	4,386,328	43,392,949
18. Net Electrical Energy (MWH)	258,885	4,139,232	41,027,344
19. Unit Service Factor	87.7	94.2	74.3
20. Unit Availability Factor	87.7	94.2	74.3
21. Unit Capacity Factor (MDC Net)	45.0	73.2	57.9
22. Unit Capacity Factor (DER Net)	43.8	71.2	56.3
23. Unit Forced Outage Rate	12.3	3.0	12.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
REFUELING, DECEMBER 31, 1980, 17 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
8	10/05/80	F	8.3	B	1				REACTOR REMAINED CRITICAL - TRIPPED TURBINE TO PLACE OFFGAS SYSTEM BACK IN SERVICE.
9	10/09/80	F	83.5	A	3				REACTOR SCRAMMED ON TURBINE STOP VALVE CLOSURE - EHC PUMP ELECTRICAL MALFUNCTION CAUSED SCRAM.

SUMMARY

DRESDEN 2 OPERATED WITH AN OUTAGE ON THE 5TH FOR MAINTENANCE AND ANOTHER OUTAGE ON THE 9TH FOR EQUIPMENT FAILURE.

LEGEND

[1] Type:
F–Forced
S–Scheduled

[2] Reason:
A–Equipment Failure
B–Maintenance or Test
C–Refueling
D–Regulatory Restriction
E–Operator Training & License Examination
F–Administrative
G–Operational Error
H–Other

[3] Method:
1–Manual
2–Manual Scram
3–Automatic Scram
4–Continuations
5–Load Reduction
9–Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

DRESDEN 3

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Illinois/Grundy 9 miles E of Morris
2.	Electric Reliability Council	Mid-America Interpool Network
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2527
5.	Design Electrical Rating (MWE Net)	794
6.	Date of Initial Criticality	January 31, 1971
7.	Date of Commercial Operation	November 16, 1971
8.	Condenser Cooling Method	Cooling Lake
9.	Condenser Cooling Water Source	Kankakee and Des Plaines Rivers

Utility & Contractor Information

10.	Licensee	Commonwealth Edison Co.
11.	Corporate Address	P. O. Box 767 Chicago, Illinois 60690
12.	Corporate Contact	Cordell Reed, Vice President
13.	Architect/Engineer	Sargent & Lundy
14.	Nuclear Steam Supply System	GE
15.	Constructor	United Engineers & Constructors, Inc.

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	R. Spessard
19.	IE Resident Inspector	T. Tongue 815-942-9267
20.	Licensing Project Manager	R. Bevan
21.	Docket Number	50-249
22.	License Number/Date of Issue	DPR-25 issued 1/12/71
23.	Public Document Room	Morris Public Library 604 Liberty Street Morris, Illinois 60451

INSPECTION STATUS

ENFORCEMENT STATUS

10 CFR 20.105 PROHIBITS POSSESSION, USE, OR TRANSFER OF LICENSED MATERIAL IN A MANNER THAT WOULD CREATE IN AN UNRESTRICTED AREA THE POTENTIAL FOR AN INDIVIDUAL TO RECEIVE A DOSE GREATER THAN TWO MILLIREMS IN ONE HOUR OR GREATER THAN 100 MILLIREMS IN SEVEN CONSECUTIVE DAYS. CONTRARY TO THE ABOVE, THE LICENSEE FOUND IN THE SCRAPYARD, AN UNRESTRICTED AREA, MATERIAL THAT PRODUCED AN EXPOSURE RATE OF APPROXIMATELY 10 MR PER HOUR AT A DISTANCE OF THREE FEET.
(8017 INFRACTION)

TECHNICAL SPECIFICATION 6.2.B REQUIRES THAT RADIATION CONTROL PROCEDURES BE MAINTAINED, MADE AVAILABLE TO ALL PERSONNEL, AND ADHERED TO. THE DOCUMENT "DRESDEN NUCLEAR STATION RADIATION CONTROL STANDARDS" IS TO PROCEDURE THAT ESTABLISHED CONTROL OVER PERSONAL RADIATION EXPOSURES. THE FOLLOWING EXAMPLES OF FAILURE TO FOLLOW THIS PROCEDURE WERE IDENTIFIED DURING THE APPRAISAL. CONTRARY TO THE ABOVE REQUIREMENTS: (A) LAUNDERED PROTECTIVE CLOTHING IN THE CLEAN LAUNDRY STORAGE BINS IN THE UNIT 1 TRACK WAY WAS FOUND READING TO 22,000 COUNTS PER MINUTE (CPM) IN CONTRAST TO THE "STANDARDS" LIMIT OF 3,000 CPM; AND (B) MATERIAL READING 10 MR PER HOUR AT THREE FEET WAS FOUND IN THE SCRAP YARD IN CONTRAST TO THE "STANDARDS" UNCONDITIONAL RELEASE LIMIT OF 100 CPM.
(8017 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

UNIT OPERATING ROUTINELY.

+LAST IE SITE INSPECTION DATE: SEPTEMBER 22-26, 1980

+ INSPECTION REPORT NO: 80-23

SUMMARY

INSPECTION SUMMARY

+ INSPECTION ON AUGUST 3 THROUGH SEPTEMBER 6, (80-21): ROUTINE, UNANNOUNCED RESIDENT INSPECTION OF OPERATIONAL SAFETY VERIFICATION; MONTHLY SURVEILLANCE OBSERVATION; MONTHLY MAINTENANCE OBSERVATION; IE BULLETIN FOLLOWUP; IE CIRCULAR FOLLOWUP; AND FOLLOWUP ON A PLANT INCIDENT. THE INSPECTION INVOLVED 283 INSPECTOR-HOURS ONSITE BY FOUR NRC INSPECTORS INCLUDING 49.5 HOURS ONSITE DURING OFF-SHIFTS. OF THE SEVEN AREAS INSPECTED, THERE WERE NO ITEMS OF NONCOMPLIANCE IDENTIFIED.

+ INSPECTION ON SEPTEMBER 15-17, (80-22): SPECIAL, UNANNOUNCED INSPECTION OF CIRCUMSTANCES PERTAINING TO ALLEGATIONS MADE BY UNIDENTIFIED INDIVIDUALS TO A CHICAGO NEWSPAPER THAT 300,000 TO 400,000 GALLONS OF WATER CONTAMINATED WITH LOW LEVELS OF RADIOISOTOPES HAD BEEN RELEASED TO THE ENVIRONMENT OVER A PERIOD OF ONE TO TWO MONTHS PRIOR TO SEPTEMBER 5, 1980; INCLUDING A REVIEW OF METHODS OF ACCOUNTING FOR INVENTORIES OF CONTAMINATED WATER; SPECIAL AND ROUTINE ENVIRONMENTAL SAMPLE RESULTS; INDEPENDENT INSPECTION OF POTENTIAL RELEASE FLOW PATHS; PROGRAMS TO DETECT LEAKAGE FROM THE STATION; AND, THE COLLECTION AND ANALYSIS OF INDEPENDENT ENVIRONMENTAL SAMPLES. THE INSPECTION INVOLVED 79 INSPECTOR-HOURS ONSITE BY NINE NRC INSPECTORS INCLUDING 11.5 INSPECTOR-HOURS ONSITE DURING OFF-SHIFTS. NO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED. ALTHOUGH IT COULD NOT BE DETERMINED WHETHER A LARGE VOLUME OF WATER DID OR DID NOT ESCAPE FROM THE STATION, THERE WAS NO DETECTABLE THREAT TO THE HEALTH AND SAFETY OF THE PUBLIC.

+ INSPECTION ON SEPTEMBER 22-26, (80-23): INCLUDED A REVIEW OF SECURITY PLAN; SECURITY PROGRAM AUDIT; SECURITY ORGANIZATION; PROTECTED AREA PHYSICAL BARRIERS; VITAL AREA PHYSICAL BARRIERS; LIGHTING; ACCESS CONTROL (IDENTIFICATION, AUTHORIZATION, AND BADGING); ACCESS CONTROL (SEARCH); ACCESS CONTROL (ESCORTING); ACCESS CONTROL (VITAL AREAS); ALARM STATIONS; DETECTION AIDS; ASSESSMENT AIDS; COMMUNICATIONS; TESTING AND MAINTENANCE; COMPENSATORY MEASURES; POWER SUPPLY; RESPONSE; LOCKS, KEYS, AND COMBINATIONS; RECORDS AND REPORTS; FOLLOWUP ON PREVIOUS ITEMS OF NONCOMPLIANCE/DEVIATIONS. THE INSPECTION INVOLVED 84 INSPECTOR-HOURS ONSITE BY TWO NRC INSPECTORS. THE INSPECTION BEGAN DURING DAY SHIFT PERIOD; 26 INSPECTOR HOURS WERE ACCOMPLISHED DURING OFF-SHIFT PERIODS. THE LICENSEE WAS FOUND TO BE IN COMPLIANCE WITH NRC REQUIREMENTS WITHIN THE ABOVE AREAS EXAMINED DURING THIS INSPECTION EXCEPT FOR THE FOLLOWING ONE ITEM. ACCESS CONTROL (IDENTIFICATION, AUTHORIZATION, AND BADGING) - LICENSEE RECORDS FAILED TO CONTAIN PROPER SCREENING DOCUMENTATION ON CERTAIN CONTRACTOR PERSONNEL.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

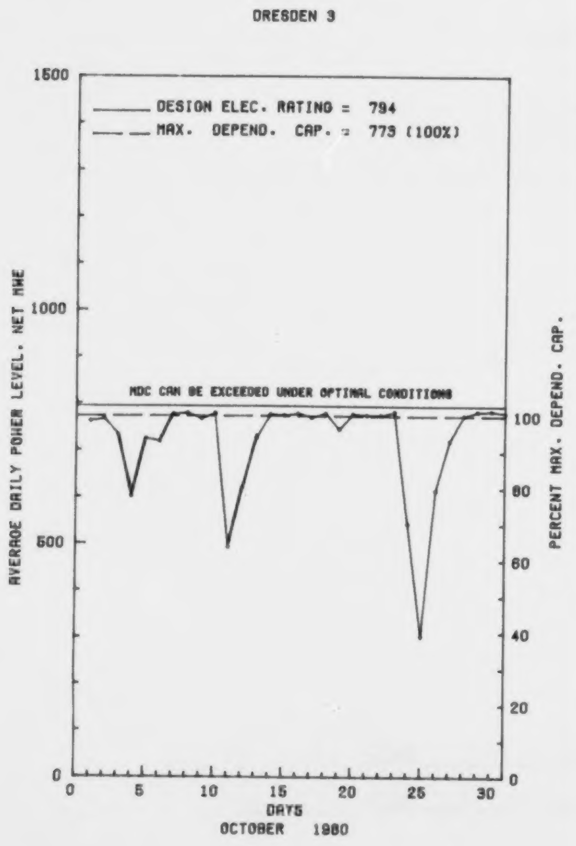
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-34/03L-0	08/31/80	09/22/80	FAILURE OF MG SET SPEED CONTROLLER CAUSED RUN BACK FROM 42% TO 22% IN RECIRCULATION PUMP SPEED.

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 12-10-80

REVIEWED BY NRR DL: John F. Hannon **DATE:** 12-11-80

DRESDEN 3

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-249
2. Utility Contact: S. A. CIESZKIEWICZ (815) 942-2920 X489
3. Licensed Thermal Power (MWt): 2527
4. Nameplate Rating (Gross MWe): 920 x 0.9 = 828
5. Design Electrical Rating (Net MWe): 794
6. Maximum Dependable Capacity (Gross MWe): 812
7. Maximum Dependable Capacity (Net MWe): 773
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	745.0	7,320.0	81,361.0
12. Hours Reactor Was Critical	736.3	5,082.7	60,848.1
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	730.2	4,985.6	58,310.2
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,720,128	11,160,381	113,432,049
17. Gross Electrical Energy (MWH)	559,450	3,571,895	37,001,569
18. Net Electrical Energy (MWH)	533,139	3,381,039	35,074,[illegible]30
19. Unit Service Factor	98.0	68.1	71.7
20. Unit Availability Factor	98.0	68.1	71.7
21. Unit Capacity Factor (MDC Net)	92.6	59.8	55.8
22. Unit Capacity Factor (DER Net)	90.1	58.2	54.3
23. Unit Forced Outage Rate	2.0	2.6	15.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
TMI MODIFICATIONS, DECEMBER 8, 1980, 4 OR 5 DAYS TENTATIVE.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
14	10/24/80	F	14.8	A	3				REACTOR SCRAMMED - ESSENTIAL SERVICE M.G. SET TRIPPED AND APRM'S UPSCALE.

SUMMARY

DRESDEN 3 OPERATED AT NEAR FULL POWER FOR MOST OF OCTOBER, WITH ONE OUTAGE ON THE 24TH FOR EQUIPMENT FAILURE.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

DUANE ARNOLD

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Iowa/Linn 8 miles NW of Cedar Rapids
2.	Electric Reliability Council	Mid-Continent Area Reliability Coordination Agreement
3.	Type of Reactor	BWR
4.	Power Level (MWT)	1658
5.	Design Electrical Rating (MWE Net)	538
6.	Date of Initial Criticality	March 23, 1974
7.	Date of Commercial Operation	February 1, 1975
8.	Condenser Cooling Method	Cooling Towers
9.	Condenser Cooling Water Source	Cedar Rapids River Circulating Water System

Utility & Contractor Information

10.	Licensee	Iowa Electric Light and Power
11.	Corporate Address	IE Towers, P. O. Box 351 Cedar Rapids, Iowa 52406
12.	Corporate Contact	D. Arnold, President
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	GE
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	W. S. LITTLE
19.	IE Resident Inspector	W. F. CHRISTIANSON
20.	Licensing Project Manager	K. T. Eccleston
21.	Docket Number	50-331
22.	License Number/Date of Issue	DPR-49 issued 2/22/74
23.	Public Document Room	Reference Service Cedar Rapids Public Library 428 Third Avenue, S.E. Cedar Rapids, Iowa 52401

INSPECTION STATUS

ENFORCEMENT STATUS

10 CFR PART 50, APPENDIX B, CRITERION V STATES IN PART, "ACTIVITIES THAT AFFECT QUALITY SHALL BE PRESCRIBED BY DOCUMENTED INSTRUCTIONS, PROCEDURES..." THE DAEC "QUALITY ASSURANCE PROGRAM BOUNDARY," QAP 1301.4, 5.2 COMMITS THE QA PROGRAM TO MEET THE REQUIREMENTS OF SAFETY GUIDE 33. SAFETY GUIDE 33 INCLUDES THE REACTOR CLEANUP SYSTEM OPERATION IN THE LIST OF SAFETY RELATED ACTIVITIES REQUIRING PROCEDURES. CONTRARY TO THE ABOVE, DURING NORMAL PLANT OPERATIONS, THE REACTOR WATER CLEANUP SYSTEM WAS OPERATED IN AN OFF NORMAL CONDITION FROM AUGUST 2, 1980 TO AUGUST 7, 1980 WITHOUT APPROVED PROCEDURES. OPERATION IN THE OFF NORMAL CONDITION WITH NO RETURN FLOW TO THE REGENERATIVE HEAT EXCHANGER, ATTRIBUTED TO 14 PRIMARY CONTAINMENT GROUP FIVE ACTUATIONS DUE TO THE ELEVATED TEMPERATURES IN THE REACTOR WATER CLEANUP SYSTEM ROOM.
(8013 DEFICIENCY)

TECHNICAL SPECIFICATIONS SECTION 4.7.A.2.D.3 STATES "TYPE C TESTS SHALL BE PERFORMED DURING EACH REACTOR SHUTDOWN FOR MAJOR REFUELING OR OTHER CONVENIENT INTERVAL BUT IN NO CASE AT INTERVALS GREATER THAN TWO YEARS." CONTRARY TO THE ABOVE, THE HPCI/RCIC EXHAUST VACUUM BREAKERS LISTED IN TECHNICAL SPECIFICATION TABLE 3.7.2, WERE NOT TESTED FROM INITIAL INSTALLATION IN JUNE 1974 TO MARCH 1980 THEREBY EXCEEDING THE TWO YEAR INTERVAL.
(8013 INFRACTION)

CONTRARY TO PARAGRAPH 1.4.1 OF THE APPROVED SECURITY PLAN, SIX WATCHMEN WERE NOT ADEQUATELY SCREENED PRIOR TO BEING GRANTED UNESCORTED ACCESS TO THE PROTECTED AREA.
(8017 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

UNIT OPERATING ROUTINELY.

+LAST IE SITE INSPECTION DATE: SEPTEMBER 8-12, 1980

+ INSPECTION REPORT NO: 80-17

SUMMARY

INSPECTION SUMMARY

+ INSPECTION ON AUGUST 1-31, (80-13): ROUTINE INSPECTIONS, REVIEW AND OBSERVATIONS OF PLANT OPERATIONS AND MAINTENANCE, PROCEDURES, DOCUMENTATION AND WORK PERFORMED. THE INSPECTION INVOLVED 201 INSPECTOR-HOURS ONSITE BY TWO NRC INSPECTORS INCLUDING 16 INSPECTOR-HOURS ONSITE DURING OFF-SHIFTS. OF THE AREAS INSPECTED, TWO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED (FAILURE TO HAVE APPROVED PROCEDURES FOR OPERATING THE RWCU SYSTEM WITHOUT RWCU PUMPS IN OPERATION; AND FAILURE TO PERFORM TYPE C LEAK RATE TESTING ON THE HPCI/RCIC EXHAUST VACUUM BREAKERS SINCE INSTALLATION.)

+ INSPECTION ON SEPTEMBER 8-12, (80-17): INCLUDED A REVIEW OF SECURITY PLAN; SECURITY PROGRAM AUDIT; SECURITY ORGANIZATION; PROTECTED AREA PHYSICAL BARRIERS; VITAL AREA PHYSICAL BARRIERS; LIGHTING; ACCESS CONTROL (IDENTIFICATION, AUTHORIZATION, AND BADGING); ACCESS CONTROL (SEARCH); ACCESS CONTROL (ESCORTING); ACCESS CONTROL (VITAL AREAS); ALARM STATIONS; DETECTION AIDS; ASSESSMENT AIDS; COMMUNICATIONS; TESTING AND MAINTENANCE; COMPENSATORY MEASURES; RESPONSE; LOCKS, KEYS, AND COMBINATIONS; RECORDS AND REPORTS; FOLLOWUP ON PREVIOUS ITEMS OF NONCOMPLIANCE/DEVIATIONS. THE INSPECTION INVOLVED 82 INSPECTOR-HOURS ONSITE BY TWO NRC INSPECTORS. THE INSPECTION BEGAN DURING AN OFF-SHIFT PERIOD; 22 INSPECTOR-HOURS WERE ACCOMPLISHED DURING OFF-SHIFT PERIODS. THE LICENSEE WAS FOUND TO BE IN COMPLIANCE WITH NRC REQUIREMENTS WITHIN THE 20 AREAS EXAMINED DURING THIS INSPECTION EXCEPT FOR THE FOLLOWING ONE ITEM: SECURITY ORGANIZATION - SOME WATCHMEN WERE NOT ADEQUATELY SCREENED IN ACCORDANCE WITH SECURITY PLAN COMMITMENTS.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

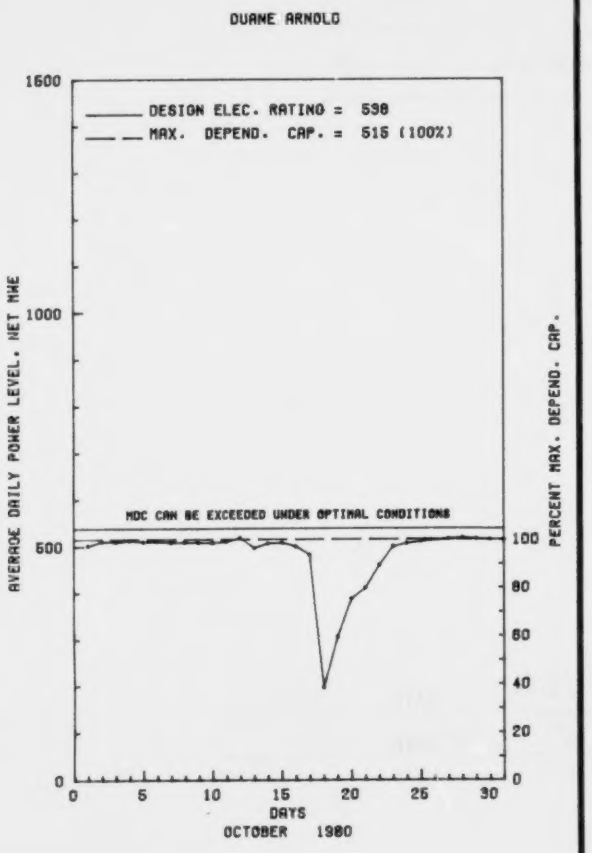
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-44/ 03L-0	08/24/80	09/23/80	ROD WITHDRAWAL BLOCK.
80-45/ 03L-0	09/08/80	10/08/80	CST LOW LEVEL SWITCH INOPERABLE.
80-47/ 03L-0	09/22/80	10/09/80	REACTOR WATER CLEANUP SYSTEM RETURN LINE WOULD NOT CLOSE WITHIN TS.
80-48/ 03L-0	09/12/80	10/09/80	RWCU PUMP PROBLEMS CAUSED RWCU TO BE TAKEN OUT OF SERVICE.
80-49/ 03L-0	09/19/80	10/09/80	HPCI LEAK DETECTION TEMP. DIFFERENTIAL SWITCH FAILED DOWNSCALE.

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 12-10-80

REVIEWED BY NRR DL: Kenneth T. Eccleston **DATE:** 12-11-80

DUANE ARNOLD

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-331
2. Utility Contact: J. Van Sickel (319) 851-5611
3. Licensed Thermal Power (MWt): 1658
4. Nameplate Rating (Gross MWe): 663 x 0.9 = 597
5. Design Electrical Rating (Net MWe): 538
6. Maximum Dependable Capacity (Gross MWe): 545
7. Maximum Dependable Capacity (Net MWe): 515
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	745.0	7,320.0	50,400.0
12. Hours Reactor Was Critical	745.0	5,364.4	35,928.5
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	745.0	5,251.7	35,370.8
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,119,900	7,100,220	44,229,780
17. Gross Electrical Energy (MWH)	381,919	2,375,971	14,800,337
18. Net Electrical Energy (MWH)	360,174	2,228,726	13,843,790
19. Unit Service Factor	100.0	71.7	69.6
20. Unit Availability Factor	100.0	71.7	69.6
21. Unit Capacity Factor (MDC Net)	93.9	59.1	53.3
22. Unit Capacity Factor (DER Net)	89.9	56.6	51.1
23. Unit Forced Outage Rate	.0	6.3	19.0

24. Shutdowns Scheduled Over Next 6 Months (Type,Date,and Duration of Each):
REFUELING, MARCH 7, 1981, - 9 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
19	10/18/80	S	.0	A	5				POWER WAS REDUCED TO ALLOW REPAIRS TO BE MADE TO A TURBINE SEAL LEAK OFF-LINE.

SUMMARY

DUANE ARNOLD OPERATED AT NEAR FULL POWER DURING OCTOBER, WITH ONE REDUCTION FOR EQUIPMENT REPAIR ON THE .8TH.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

FARLEY 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Alabama/Houston 24 miles SE of Dothan
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2652
5.	Design Electrical Rating (MWE Net)	829
6.	Date of Initial Criticality	August 9, 1977
7.	Date of Commercial Operation	December 1, 1977
8.	Condenser Cooling Method	Once Through With Forced Draft Cooling Towers
9.	Condenser Cooling Water Source	Chattahoochee River

Utility & Contractor Information

10.	Licensee	Alabama Power Company
11.	Corporate Address	600 North 18th Street Birmingham, Alabama
12.	Corporate Contact	F. L. Clayton, Jr. Senior Vice President (205-323-5341)
13.	Architect/Engineer	Bechtel/SSI
14.	Nuclear Steam Supply System	Westinghouse
16.	Constructor	Daniel

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. Lewis
18.	IE Section Chief	R. D. Martin
19.	IE Resident Inspector	W. H. Bradford (205-899-3386)
20.	Licensing Project Manager	E. A. Reeves
21.	Docket Number	50-348
22.	License Number/Date of Issue	NPF-2, 6/25/77
23.	Public Document Room	G. S. Houston Memorial Library 212 W. Berdeshaw Street Dothan, Alabama 36301

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO THE REQUIREMENTS OF CRITERION V OF APPENDIX B TO 10CFR50, THE LICENSEE DID NOT HAVE A PROCEDURE WHICH ADEQUATELY ADDRESSED THE PREPARATION OF AS BUILT DRAWINGS FOR MASONRY WALLS.
(8026 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

+ REFUELING OUTAGE SCHEDULED TO BEGIN NOVEMBER 7, 1980.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

+ FULL POWER OPERATION.

+ LAST IE SITE INSPECTION DATE: OCTOBER 1-31, 1980

+ INSPECTION REPORT NO: 50-348/80-35

SUMMARY

INSPECTION SUMMARY

+ INSPECTION SEPTEMBER 10-12 (80-26): THIS SPECIAL, ANNOUNCED INSPECTION INVOLVED 9 INSPECTOR-HOURS ON SITE IN THE AREAS OF IEB 80-11, AND UNIT 1 CONTAINMENT TENDON SURVEILLANCE QUALITY RECORDS. OF THE TWO AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN TWO AREAS; ONE ITEM OF NONCOMPLIANCE WAS FOUND IN ONE AREA (INFRACTION - INADEQUATE PROCEDURE FOR SURVEY OF CONCRETE MASONRY WALLS).

INSPECTION OCTOBER 6-10 (80-34): THIS ROUTINE ANNOUNCED PREOPERATIONAL INSPECTION INVOLVED 33 INSPECTOR-HOURS ON SITE IN THE AREAS OF EMERGENCY PLANNING, INCLUDING COORDINATION WITH OFFSITE SUPPORT GROUPS, EMERGENCY FACILITIES EQUIPMENT AND PROCEDURES, EMERGENCY TRAINING, FOLLOW-UP ON PREVIOUS INSPECTION FINDINGS AND FOLLOW-UP ON IE BULLETIN. OF THE FIVE AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

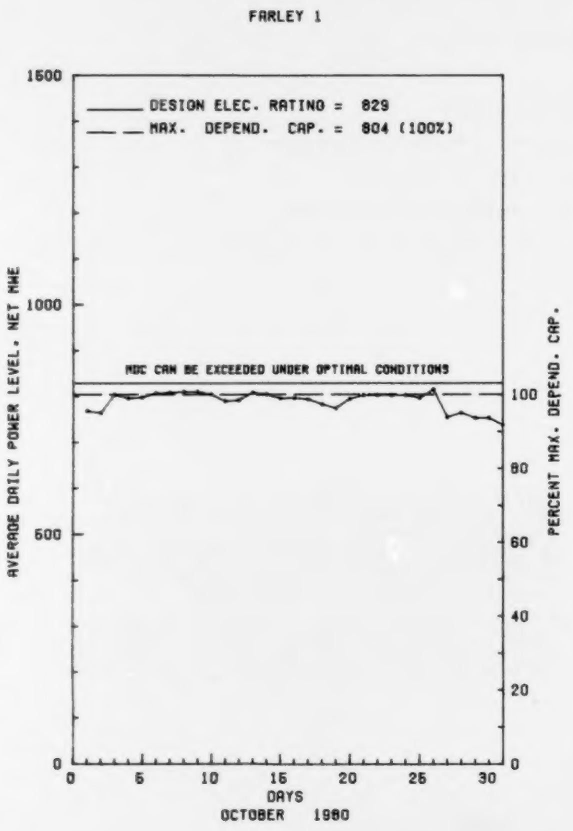
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-048/ 03L-0	08/17/80	09/16/80	CONTAINMENT ATMOSPHERE PARTICULATE AND GASEOUS RADIOACTIVITY MONITORING SYSTEMS WERE INOPERABLE.
80-049/ 03L-0	08/20/80	09/18/80	REACTOR COOLANT SYSTEM LOOP 1C FLOW TRIP WAS INOPERABLE DUE TO DEFECTIVE TRANSISTORS.
80-050/ 03L-0	08/25/80	09/24/80	SERVICE WATER TRAIN B DC DISTRIBUTION SYSTEM WAS INOPERABLE.
80-051/ 03L-0	08/26/80	09/25/80	DIESEL GENERATOR 1C WAS INOPERABLE DUE TO A FUEL OIL SYSTEM PROBLEM.
80-052/ 03L-0	08/29/80	09/26/80	SEQUENCER BIN FAILED TO PICKUP CERTAIN LOADS DURING TESTING DUE TO SETPOINT DRIFT.

REVIEWED BY OMPA LOEB: Bobe **DATE:** 12/10/80

REVIEWED BY NRR DL: Ed A. Reeves **DATE:** 12/11/80

FARLEY 1

AVERAGE POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-348
2. Utility Contact: W. G. Hairston,III (205) 899-5156
3. Licensed Thermal Power (MWt): 2652
4. Nameplate Rating (Gross MWe): 1045 x 0.85 = 888
5. Design Electrical Rating (Net MWe): 829
6. Maximum Dependable Capacity (Gross MWe): 845
7. Maximum Dependable Capacity (Net MWe): 804
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	745.0	7,320.0	25,584.0
12. Hours Reactor Was Critical	745.0	6,077.8	17,013.3
13. Reactor Reserve Shutdown Hours	.0	1,041.5	2,446.8
14. Hours Generator On-Line	745.0	5,944.9	16,535.4
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,933,931	15,009,306	41,607,321
17. Gross Electrical Energy (MWH)	619,065	4,772,706	13,296,744
18. Net Electrical Energy (MWH)	587,728	4,497,174	12,501,182
19. Unit Service Factor	100.0	81.2	64.6
20. Unit Availability Factor	100.0	81.2	64.6
21. Unit Capacity Factor (MDC Net)	98.1	76.4	60.8
22. Unit Capacity Factor (DER Net)	95.2	74.1	58.9
23. Unit Forced Outage Rate	.0	6.3	7.4

24. Shutdowns Scheduled Over Next 6 Months (Type,Date,and Duration of Each):
REFUELING OUTAGE; NOVEMBER 7, 1980; APPROX. 10 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
071	09/28/80	F	.0	B	5		HC	ZZZZZZ	CONTINUATION OF POWER REDUCTION FROM 9/30/80 TO INVESTIGATE LOW CONDENSER VACUUM. UNIT POWER HELD AT 640 MWE.

SUMMARY

FARLEY 1 OPERATED NORMALLY DURING OCTOBER, WITH A POWER REDUCTION CONTINUING FROM SEPTEMBER FOR LOW CONDENSER VACUUM.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

FITZPATRICK

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	New York/Oswego 8 miles NE of Oswego
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2436
5.	Design Electrical Rating (MWE Net)	821
6.	Date of Initial Criticality	November 17, 1974
7.	Date of Commercial Operation	July 28, 1975
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Lake Ontario

Utility & Contractor Information

10.	Licensee	Power Authority of the State of New York
11.	Corporate Address	10 Columbus Circle New York, New York 10019
12.	Corporate Contact	J. P. Bayne, Manager of Nuclear Operations
13.	Architect/Engineer	Stone & Webster
14.	Nuclear Steam Supply System	GE
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Section Chief	H. B. KISTER
19.	IE Resident Inspector	W. H. BAUNACK (315) 342-4907 FTS: 264-3311
20.	Licensing Project Manager	P. Polk (X7872)
21.	Docket Number	50-333
22.	License Number/Date of Issue	DPR-59, 10/17/74
23.	Public Document Room	State University College of Oswego Penfield Library - Documents Oswego, NY 13126 (315)341-2323

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO TECHNICAL SPECIFICATIONS AND ADMINISTRATIVE PROCEDURES CONTROL ROOM AND SHIFT SUPERVISOR'S COPIES OF OPERATING PROCEDURES MANUALS WERE NOT KEPT UP TO DATE WITH CURRENT REVISIONS.
(8012 DEFICIENCY)

TECHNICAL SPECIFICATION 6.8 STATES: "WRITTEN PROCEDURES...SHALL BE ESTABLISHED, IMPLEMENTED AND MAINTAINED FOR THE FIRE PROTECTION PROGRAM." LICENSEE'S MAINTENANCE PROCEDURE NO. 57.5, "PROCEDURE FOR OPENING AND SEALING ELECTRICAL CONDUIT SLEEVES," REVISION 5, JUNE 23, 1980, PARAGRAPH 8.1, STATES: "IN PROGRESS INSPECTION WILL BE CONDUCTED IN ACCORDANCE WITH TECH-SIL QUALITY CONTROL MANUAL (NP 12-0-101)...." TECH-SIL QUALITY CONTROL MANUAL NP 12-0-101, SECTION 210, PARAGRAPH 1.4, STATES: "RECORD DAM APPROVAL ON FORM TS-QA-5C FOR EACH PENETRATION FILLED (WHERE DAMS ARE TO REMAIN IN PLACE) ALL PENETRATIONS ABOVE 4 INCHES." CONTRARY TO THE ABOVE, NOT ALL PENETRATIONS REQUIRING PERMANENT DAMMING WERE INSPECTED, AND THE INSPECTED PENETRATIONS WERE NOT IDENTIFIED ON FORM TS-QA-5C.
(8017 INFRACTION)

TECHNICAL SPECIFICATION 6.8 STATES: "WRITTEN PROCEDURES...SHALL BE ESTABLISHED, IMPLEMENTED AND MAINTAINED FOR THE FIRE PROTECTION PROGRAM." LICENSEE'S PROCEDURE NO. 10.1.1, PROCEDURE FOR CONTROL OF MAINTENANCE, REVISION 2, JULY 1979, SECTION 7.2, STATES: "THE WORK TRACKING FORM...SHALL BE ISSUED...TO PROPERLY...TRACK, CONTROL AND DOCUMENT CORRECTIVE MAINTENANCE FOR CATEGORY 1, SAFETY RELATED OR QC INSPECTION REQUIRED WORK." THE LICENSEE'S WORK REQUEST EVENT DEFICIENCY (WRED) NO. 6979 ISSUED MAY 20, 1980 IDENTIFIES THE ELECTRICAL CABLE PENETRATION MODIFICATION FI-80-07A AS CATEGORY 1, SAFETY RELATED AND QC INSPECTION REQUIRED. CONTRARY TO THE ABOVE, THE LICENSEE HAS COMPLETED ELECTRICAL PENETRATION SEALING WORK THROUGH THE SECONDARY CONTAINMENT AT ELEVATION 272' AND BETWEEN THE CABLE SPREADING ROOM AND THE BATTERY ROOM, WITHOUT ISSUING APPROPRIATE WORK TRACKING FORMS.
(8017 INFRACTION)

TECHNICAL SPECIFICATION 3.12F, PARAGRAPH A, STATES: "THE FIRE BARRIER PENETRATION SEALS SHALL BE FUNCTIONAL FOR EACH PROTECTED AREA...." TECHNICAL SPECIFICATION 3.12.F, PARAGRAPH 6 STATES: "WITHIN ONE HOUR THAT ANY OF THE FIRE BARRIER PENETRATION SEALS FOR ANY PROTECTED AREA IS FOUND OR MADE NONFUNCTIONAL A CONTINUOUS FIRE WATCH SHALL BE ESTABLISHED ON ONE SIDE OF THE FIRE BARRIER." CONTRARY TO THE ABOVE, THE FIRE DOOR BETWEEN THE EAST AND WEST CABLE TUNNEL AND THE FIRE DOOR BETWEEN THE WEST CABLE TUNNEL AND ELECTRIC FIRE PUMP ROOM WERE FOUND OPEN AND UNATTENDED ON SEPTEMBER 9, 1980 WITHOUT ADEQUATE MEASURES EITHER ESTABLISHED OR IMPLEMENTED TO ENSURE THAT A FIRE WATCH WAS IN EFFECT.
(8017 INFRACTION)

THE INSPECTOR REVIEWED AMENDMENT NO. 47 TO FACILITY OPERATING LICENSE NO. DPR-59. IN SECTION 6 OF THE DOCUMENT, ADMINISTRATIVE CONTROLS, THE LICENSEE PROPOSED TO AMEND THE EXISTING FIRE PROTECTION ADMINISTRATIVE PROGRAM TO CONFORM TO RECOMMENDATIONS PRESENTED IN THE NRC'S GUIDANCE DOCUMENT, "NUCLEAR PLANT FIRE PROTECTION FUNCTIONAL RESPONSIBILITIES, ADMINISTRATIVE CONTROLS AND QUALITY ASSURANCE," INCLUDING CONTROLS OVER COMBUSTIBLES. THE INSPECTOR REVIEWED THE LICENSEE PROCEDURES: HOUSEKEEPING AND CLEANLINESS CONTROL, NO. 10.1.7, MAY 5, 1980; CONTROL OF COMBUSTIBLES AND FLAMMABLE MATERIAL, NO. 10.1.10, DECEMBER 19, 1979; AND IDENTIFIED THAT THE FOLLOWING NRC GUIDELINES WERE NOT INCORPORATED IN THE LICENSEE'S PROCEDURES: THE REMOVAL OF ALL WASTE, DEBRIS, SCRAPS, RANGE, OIL SPILLS, OR OTHER COMBUSTIBLES RESULTING FROM THE WORK ACTIVITY IN THE AREA FOLLOWING COMPLETION OF THE ACTIVITY OR AT THE END OF EACH WORK SHIFT, WHICHEVER IS SOONER; PERIODIC INSPECTION FOR ACCUMULATION OF COMBUSTIBLES.
(8017 DEVIATION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

MANAGERIAL ITEMS

- EFFECTIVE SEPTEMBER 15, 1980 AN ORGANIZATION CHANGE WAS MADE WHICH HAS THE SENIOR VICE PRESIDENT - NUCLEAR GENERATION REPORT TO THE PRESIDENT AND CHIEF OPERATING OFFICER.

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-070/ 03L	08/10/80	09/08/80	IRM FAILURES DURING STARTUP
80-071/ 03L	08/10/80	09/09/80	RCIC PROBLEMS IDENTIFIED DURING STARTUP
80-072/ 03L	08/12/80	09/11/80	LPCI INVERTER TRIPS
80-073/ 03L	08/15/80	09/12/80	B OFF GAS MONITOR FOUND OUT OF CALIBRATION
80-074/ 03L	09/03/80	10/01/80	HIGH DRYWELL PRESSURE SWITCH DRIFT IDENTIFIED DURING SURVEILLANCE TESTING.
80-075/ 03L	09/13/80	10/10/80	B LPCI INVERTER FAILED LOGIC CARD A2
80-076/ 04T	10/02/80	10/15/80	ACTIVITY IN AN AQUATIC MOSS (PERIPHYTON) GREATER THAN 10 TIMES CONTROL SAMPLE.

PLANT STATUS:

- THE PLANT WAS OPERATED CONTINUOUSLY AT FULL POWER WITH THE EXCEPTION OF A FOUR DAY OUTAGE RESULTING FROM A REACTOR TRIP CAUSED WHEN A WORKMAN DROPPED A PIECE OF SCAFFOLDING ON TO THE RPS M-G SET. REPRESENTATIVE STACK AND OFFGAS RELEASE RATES AT FULL POWER WERE 5,500 AND 15,000 MICROCURIES PER SECOND. THE RECOMBINER IS OUT OF SERVICE.

- LAST IE SITE INSPECTION DATE: 10/20-24/80

- INSPECTION REPORT NO: 50-333/80-19

SUMMARY

INSPECTION SUMMARY

- 50-333/80-12 - JUL 1 - AUG 29: ROUTINE INSPECTION BY RESIDENT INSPECTORS (124 HRS) OF LICENSEE ACTION ON PREVIOUS INSPECTION FINDINGS, OPERATIONAL SAFETY VERIFICATION, OBSERVATION OF PHYSICAL SECURITY, MONTHLY MAINTENANCE OBSERVATION, INSPECTOR WITNESSING OF SURVEILLANCE TESTS, PREPARATION FOR PLANT START-UP FOLLOWING REFUELING, PLANT STARTUP FOLLOWING REFUELING, LICENSEE ACTIONS TAKEN IN RESPONSE TO IE BULLETIN 80-17, OPERATION EVENT, VERIFYING LICENSEE APPLICATION SUBMITTALS FOR LICENSEE TRAINING STAFF PERSONNEL, LOCATION OF LOAD CENTERS, AND MAIN STEAM ISOLATION VALVE LEAK RATE TESTING. ONE NONCOMPLIANCE WAS IDENTIFIED: FAILURE TO ADEQUATELY CONTROL WRITTEN PROCEDURES.

- 50-333/80-16 - SEP 8-12: ROUTINE, ONSITE, UNANNOUNCED INSPECTION BY TWO REGION-BASED INSPECTORS (50 HRS) OF FOLLOWUP ON PRIOR IDENTIFIED ITEMS; ADMINISTRATIVE CONTROLS FOR FACILITY PROCEDURES; CONFORMANCE TO TECHNICAL SPECIFICATIONS; VERIFICATION THAT TEMPORARY AND PERMANENT PROCEDURE CHANGES ARE MADE IN CONFORMANCE TO TECHNICAL SPECIFICATION REQUIREMENTS AND LICENSEE PROCEDURES; VERIFICATION THAT PROCEDURE CHANGES WERE IN CONFORMANCE TO 10 CFR 50.59(A) AND (B) REQUIREMENTS; CHECKLISTS AND RELATED FORMS REFLECTED LATEST CHANGES; CONTROL ROOM AND FACILITY TOURS; AND OBSERVATIONS OF AN EMERGENCY DRILL. NO NONCOMPLIANCES WERE IDENTIFIED.

- 50-333/80-17 - SEP 8-11: ROUTINE, UNANNOUNCED INSPECTION BY ONE REGION-BASED INSPECTOR (16 HRS) OF THE FACILITY'S FIRE BARRIER PENETRATIONS MODIFICATION INCLUDING THE INSTALLATION OF PIPING PENETRATION SEALS; ELECTRICAL PENETRATION SEALS; OBSERVATION OF WORK IN-PROGRESS; REVIEW OF PROCEDURES, DRAWINGS AND MANUALS; AND THE OBSERVATION OF CRITICAL FIRE AREAS. THREE NONCOMPLIANCES WERE IDENTIFIED: FAILURE TO ESTABLISH FIRE WATCH; FAILURE TO FOLLOW FIRE PROTECTION PROCEDURES; AND, FAILURE TO ISSUE APPROPRIATE WORK TRACKING FORMS.

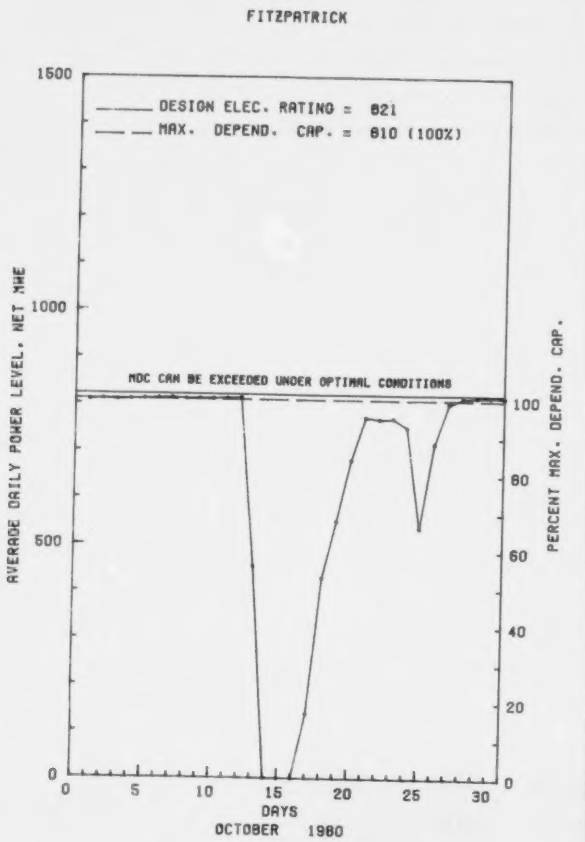
50-333/80-18 - SEP 22-26: SPECIAL UNANNOUNCED INSPECTION BY A REGION-BASED INSPECTOR AND AN NRC HQ OBSERVER (22 HRS) OF THE LICENSEE'S PHYSICAL SECURITY PROGRAM RELATIVE TO THE NEWLY FORMED CONTINGENCY SECURITY FORCE ORGANIZATION AND RELATED RECORDS AND REPORTS. FOLLOWUP ON LICENSEE ACTION ON PREVIOUS ITEMS OF NONCOMPLIANCE. NO NONCOMPLIANCES WERE IDENTIFIED.

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 12-10-80

REVIEWED BY NRR DL: **DATE:** 12/11/80

FITZPATRICK

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-333
2. Utility Contact: E. Zufelt (315) 342-3840
3. Licensed Thermal Power (MWt): 2436
4. Nameplate Rating (Gross MWe): 981 x 0.9 = 883
5. Design Electrical Rating (Net MWe): 821
6. Maximum Dependable Capacity (Gross MWe): 830
7. Maximum Dependable Capacity (Net MWe): 810

ITEM 21 CALCULATED WITH A WEIGHTED MDC

8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: MDC NET CHANGED FROM 800 TO 810 MWE ON 10/26/80
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	745.0	7,320.0	46,153.0
12. Hours Reactor Was Critical	664.7	4,819.8	31,450.0
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	652.8	4,700.8	30,438.8
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,456,296	9,496,392	60,684,586
17. Gross Electrical Energy (MWH)	503,920	3,252,440	20,606,140
18. Net Electrical Energy (MWH)	488,985	3,149,280	19,939,855
19. Unit Service Factor	87.6	64.2	66.0
20. Unit Availability Factor	87.6	64.2	66.0
*21. Unit Capacity Factor (MDC Net)	81.8	53.8	58.7
22. Unit Capacity Factor (DER Net)	79.9	52.4	52.6
23. Unit Forced Outage Rate	12.4	5.2	18.9

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): SCHED SNUBBER INSPECT & MAINT - JANUARY, 1981-APPROX 10 DAYS
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type¹	Duration (Hours)	Reason²	Method of Shutting Down Reactor³	Licensee Event Report No.	System Code⁴	Component Code⁵	Cause & Corrective Action to Prevent Recurrence
17	10/13/80	F	92.2	A	3		IA	GENERA	LOSS OF "B" RPS MG SET.
18	10/24/80	S	.0	H	5				POWER REDUCED FOR CONTROL ROD PATTERN CHANGE.

SUMMARY

FITZPATRICK OPERATED AT NEAR 100% OF RATED THERMAL POWER FOR THIS REPORTING PERIOD. ON OCTOBER 13, 1980, THE UNIT SCRAMMED DUE TO LOSS OF "B" RPS MG SET, AND A REDUCTION FOR CONTROL ROD PATTERN CHANGE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

FORT CALHOUN

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Nebraska/Washington 19 miles N of Omaha
2.	Electric Reliability Council	Mid-Continent Area Reliability Coordination Agreement
3.	Type of Reactor	PWR
4.	Power Level (MWT)	1500
5.	Design Electrical Rating (MWE Net)	490
6.	Date of Initial Criticality	August 6, 1973
7.	Date of Commercial Operation	June 20, 1974
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Missouri River

Utility & Contractor Information

10.	Licensee	W. C. Jones Division Manager – Production Operations Omaha Public Power District
11.	Corporate Address	1623 Harney Street Omaha, Nebraska 68102
12.	Corporate Contact	T. E. Short Assistant General Manager
13.	Architect/Engineer	Gibbs & Hill, Inc.
14.	Nuclear Steam Supply System	Combustion Engineering
15.	Constructor	Gibbs & Hill, Durham & Richardson, Inc.

Regulatory Information

16.	IE Region responsible	IV
17.	IE Branch Chief	G. L. Madsen
18.	IE Section Chief	D. M. Hunnicutt
19.	IE Resident Inspector	D. Kelley
20.	Licensing Project Manager	P. C. Wagner
21.	Docket Number	50-285
22.	License Number/Date of Issue	DPR-40, 5/24/73
23.	Public Document Room	W. Dale Clark Library 215 S. 15th Street Omaha, Nebraska 68102

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

FACILITY ITEMS (PLANS AND PROCEDURES):

MANAGERIAL ITEMS:

PLANT STATUS:

NO SIGNIFICANT INFORMATION

LAST IE SITE INSPECTION DATE: SEPTEMBER 30 - OCTOBER 3, 1980 •

INSPECTION REPORT NO: 50-285/80-18 •

SUMMARY

INSPECTION SUMMARY

• APPRAISAL ON SEPTEMBER 15-26, 1980 (80-16): ANNOUNCED APPRAISAL OF THE HEALTH PHYSICS PROGRAM ON SEPTEMBER 15-26, 1980, INCLUDING ORGANIZATION AND MANAGEMENT, PERSONNEL SELECTION, QUALIFICATION AND TRAINING, EXTERNAL AND INTERNAL EXPOSURE CONTROLS, RADIATION SURVEYS, ACCESS AND CONTAMINATION CONTROLS, RADIOACTIVE WASTE MANAGEMENT, ALARA, FACILITIES AND EQUIPMENT, AND EMERGENCY RESPONSE CAPABILITIES. SIGNIFICANT WEAKNESSES WERE IDENTIFIED IN THE AREAS OF ORGANIZATION AND STAFFING, PERSONNEL SELECTION, QUALIFICATIONS AND TRAINING, EXTERNAL EXPOSURE CONTROLS, INTERNAL EXPOSURE CONTROLS, AND RADIATION SURVEYS. THREE APPARENT ITEMS OF NONCOMPLIANCE WERE FOUND (INFRACTION - QUALIFICATION OF STAFF NOT IN ACCORDANCE WITH TECHNICAL SPECIFICATIONS; DEFICIENCY - FAILURE TO LABEL CONTAINERS CONTAINING RADIOACTIVE MATERIAL; DEFICIENCY - FAILURE TO REPORT THE EXPOSURE OF WORKERS TO THE COMMISSION.

INSPECTION CONDUCTED DURING PERIOD OF SEPTEMBER 1-30, 1980 (80-17): ROUTINE, ANNOUNCED INSPECTION INCLUDING OPERATIONAL SAFETY VERIFICATION, SURVEILLANCE TEST, MAINTENANCE ACTIVITIES, LER FOLLOW-UP, IEB FOLLOW-UP AND PLANT TRIPS. THE INSPECTION INVOLVED 165 INSPECTOR-HOURS ON-SITE BY ONE NRC INSPECTOR. WITHIN THE SIX AREAS INSPECTED NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION CONDUCTED ON SEPTEMBER 10 - OCTOBER 3, 1980 (80-18): ROUTINE, UNANNOUNCED INSPECTION OF THE DOCUMENT CONTROL PROGRAM AND SURVEILLANCE OF PIPE SUPPORT AND RESTRAINT SYSTEMS. THE INSPECTION INVOLVED THIRTY-NINE (39) INSPECTOR-HOURS ON-SITE BY TWO (2) NRC INSPECTORS. WITHIN THE TWO AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE NOTED.

REPORTS RECEIVED FROM LICENSEE

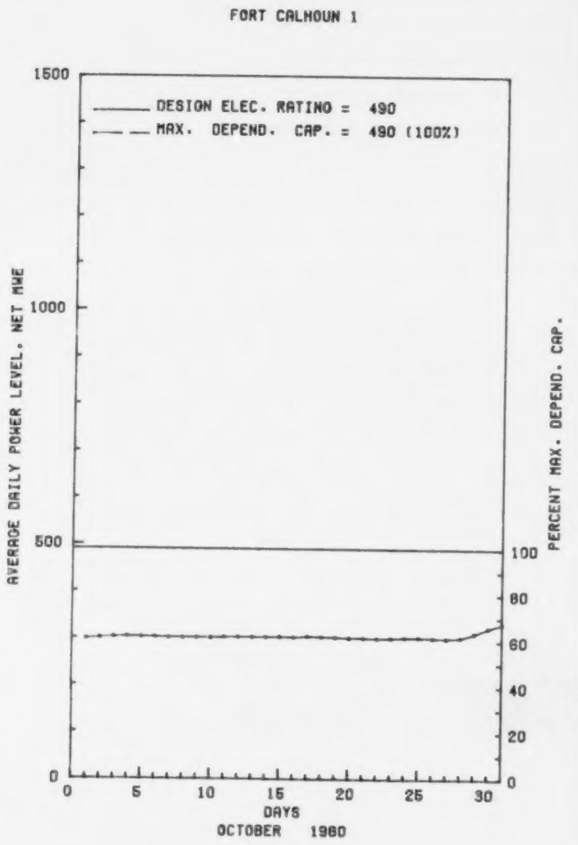
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-20 03L-0	9/5/80	9/8/80	AXIAL POWER DISTRIBUTION SETPOINT ON "D" CHANNEL OUT OF TOLERANCE
80-21 03L-0	9/10/80	9/11/80	TRANSFORMER (T4) OF DIESEL GENERATOR DG-1 FAILED
80-22 03L-0	9/11/80	9/11/80	DURING PERFORMANCE OF SI TEST A LOCKOUT RELAY FAILED TO TRIP
80-23 03L-0	9/18/80	9/26/80	NORMALLY CLOSED SET OF CONTACTS ON CV HIGH PRESSURE GAVE INCORRECT SIGNAL
80-24 03L-0	9/26/80	10/21/80	CHARGING PUMP "A" BECAME INOPERABLE WHILE CHARGING PUMP "B" WAS OUT-OF-SERVICE

REVIEWED BY OMPA LOEB: MR Beebe **DATE:** 12/10/80

REVIEWED BY NRR DL: PC Wagner **DATE:** 12/11/80

FORT CALHOUN

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-285
2. Utility Contact: T. L. PATTERSON (402) 536-4413
3. Licensed Thermal Power (MWt): 1500
4. Nameplate Rating (Gross MWe): 591 X 0.85 = 502
5. Design Electrical Rating (Net MWe): 490
6. Maximum Dependable Capacity (Gross MWe): 513
7. Maximum Dependable Capacity (Net MWe): 490

ITEMS 21 & 22 CALCULATED WITH WEIGHTED VALUES

8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: RATINGS INCREASED WITH STRETCH POWER FROM 1420 TO 1500 MWHT
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	745.0	7,320.0	62,257.0
12. Hours Reactor Was Critical	745.0	3,912.7	48,314.3
13. Reactor Reserve Shutdown Hours	.0	.0	1,309.5
14. Hours Generator On-Line	745.0	3,848.8	47,299.9
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	734,262	4,836,556	58,071,583
17. Gross Electrical Energy (MWH)	242,270	1,575,450	19,235,680
18. Net Electrical Energy (MWH)	226,440	1,489,981	18,177,418
19. Unit Service Factor	100.0	52.6	76.0
20. Unit Availability Factor	100.0	52.6	76.0
21. Unit Capacity Factor (MDC Net)	62.0	44.2	64.2
22. Unit Capacity Factor (DER Net)	62.0	44.2	63.9
23. Unit Forced Outage Rate	.0	.3	6.2

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	None								

SUMMARY

FORT CALHOUN 1 OPERATED AT 65% REACTOR POWER TO CONSERVE FUEL FROM 10/1 THRU 10/29, WITH NO REPORTABLE OUTAGES OR REDUCTIONS DURING OCTOBER.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B- Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

FORT ST. VRAIN

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Colorado/Weld 35 miles N of Denver (near Platteville)
2.	Electric Reliability Council	Western Systems Coordinating Council
3.	Type of Reactor	HTGR
4.	Power Level (MWT)	842
5.	Design Electrical Rating (MWE Net)	330
6.	Date of Initial Criticality	January 31, 1974
7.	Date of Commercial Operation	July 1, 1979
8.	Condenser Cooling Method	Towers
9.	Condenser Cooling Water Source	South Platte River/Shallow Wells

Utility & Contractor Information

10.	Licensee	Public Service Company of Colorado
11.	Corporate Address	P. O. Box 840 Denver, Colorado 80201
12.	Corporate Contact	C. K. Millen Senior Vice President
13.	Architect/Engineer	Sargent & Lundy
14.	Nuclear Steam Supply System	General Atomic Company
15.	Constructor	General Atomic Company

Regulatory Information

16.	IE Region responsible	IV
17.	IE Branch Chief	G. L. Madsen
18.	IE Section Chief	T. F. Westerman
19.	IE Resident Inspector	M. W. Dickerson (303) 785-2281
20.	Licensing Project Manager	G. Kuzmycz
21.	Docket Number	50-267
22.	License Number/Date of Issue	DPR-34, 12/21/73
23.	Public Document Room	Greeley Public Library City Complex Building Greeley, Colorado 80631

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO TS, LCO 4.1.9, THE CORE REGION DELTA TEMPERATURE EXCEEDED THE SPEDIFIED TEMPERATURES FOR UN EQUAL CORE FLOW BETWEEN 3/11/80 AND 3/12/80.
(8006 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

AREAS RELATED TO POTENTIAL EQUIPMENT MODIFICATION, WHICH ARE UNDER REVIEW AND DISCUSSION WITH THE NRC INCLUDE: HELIUM CIRCULATOR AUXILIARIES; ACCIDENT ANALYSIS RELATING TO OUTLET TEMPERATURE DISPERSION AND ORIFICE SETTINGS; HANGERS FOR 2" AND SMALLER PIPING; AND PLANT OSCILLATIONS.

FACILITY ITEMS (PLANS AND PROCEDURES):

MANAGERIAL ITEMS:

PLANT STATUS:

PLANT OPERATING AT APPROXIMATELY 50% POWER *

LAST IE SITE INSPECTION DATE: SEPTEMBER 30, 1980 *

INSPECTION REPORT NO: 50-267/80-19 *

SUMMARY

INSPECTION SUMMARY

* INSPECTION CONDUCTED ON SEPTEMBER 1-31, 1980 (80-19): ROUTINE, ANNOUNCED INSPECTION OF FOLLOW-UP ON INSPECTOR IDENTIFIED PROBLEMS, UNRESOLVED ITEMS; FOLLOW-UP ON PREVIOUS ITEMS OF NONCOMPLIANCE; SURVEILLANCE; MAINTENANCE; OPERATIONAL SAFETY VERIFICATION; REVIEW OF PERIODIC AND SPECIAL REPORTS; AND REVIEW IE BULLETINS/CIRCULARS. WITH THE SEVEN (7) AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED.

NRC POWER RESTRICTIONS: Restricted to 70% of Licensed Thermal Power.

REPORTS RECEIVED FROM LICENSEE

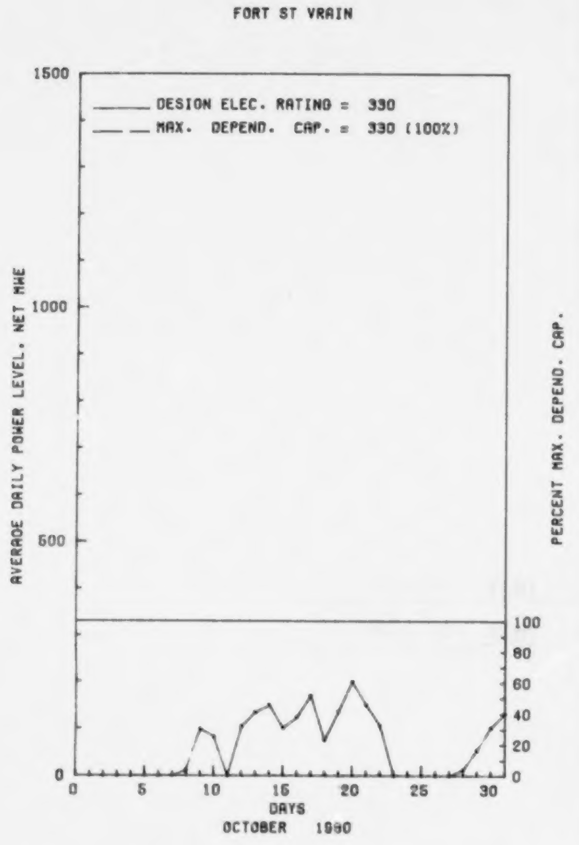
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-48 03L-0	9/1/80	10/1/80	6 STEAM PIPE RUPTURES UNDER PCRV LOOP INOPERABLE
80-49 03L-0	9/4/80	10/1/80	SET POINT OF PCRV RUPTURE DISC ASSEMBLY FOUND TO BE 826 PSIG
80-50 03L-0	9/15/80	10/6/80	2 HYDRAULIC SNUBBERS INSTALLED ON CLASS I PIPING NOT IDENTIFIED AS CLASS I
80-51 03L-0	9/22/80	10/1/80	HE CIRC SEAL MALFUNCTION PRESSURE SWITCHES, 5 OF 12 SWITCHES TRIPPED
80-52 01L-0	9/25/80	10/9/80	RADIOACTIVE LIQUID EFFLUENT DISCHARGE IN EXCESS OF VALUES SPECIFIED
80-53 03L-0	9/26/80	10/24/80	AUTOMATIC 1 B LOAD PROGRAMMING SEQUENCE DID NOT OPERATE
80-54 03L-0	9/26/80	10/24/80	8 SNUBBERS INOPERABLE
80-55 03L-0	10/1/80	10/24/80	SAMPLING REQUIREMENT OF SR NR 1.1 NOT MET
80-57 03L-0	10/4/80	10/31/80	PCRV PRESSURE GREATER THAN PENETRATION INTERSPACE PRESSURE
80-58 03L-0	10/9/80	10/31/80	POWER ON WITH EMERGENCY FEEDWATER HEADER SUPPLY TO LOOP 1 CIRCULATOR

REVIEWED BY OMPA LOEB: PE Bobe **DATE:** 12/10/80

REVIEWED BY NRR DL: **DATE:** 12/11/80

FORT ST. VRAIN

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-267
2. Utility Contact: J. W. GAHM (303) 785-2253
3. Licensed Thermal Power (MWt): 842
4. Nameplate Rating (Gross MWe): 403 x 0.85 = 343
5. Design Electrical Rating (Net MWe): 330
6. Maximum Dependable Capacity (Gross MWe): 342
7. Maximum Dependable Capacity (Net MWe): 330
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): 231
10. Reasons for Restrictions, If Any: NRC RESTRICT. 70% PEND. RESOL. OF TEMP. FLUCTUATIONS

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	745.0	7,320.0	11,737.0
12. Hours Reactor Was Critical	739.0	5,157.0	7,689.8
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	394.7	3,369.6	4,351.8
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	178,060	1,529,275	2,007,230
17. Gross Electrical Energy (MWH)	50,335	479,464	620,260
18. Net Electrical Energy (MWH)	45,246	438,625	562,209
19. Unit Service Factor	53.0	46.0	37.1
20. Unit Availability Factor	53.0	46.0	37.1
21. Unit Capacity Factor (MDC Net)	18.4	18.2	14.5
22. Unit Capacity Factor (DER Net)	18.4	18.2	14.5
23. Unit Forced Outage Rate	30.0	30.1	43.0

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): MAINT. & SURV. TESTING - MARCH, 1981, APPROX. 3 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-17	08/29/80	S	180.8	H	4				GENERATOR REMAINED OFF-LINE FOLLOWING COMPLETION OF MAINTENANCE AND SURVEILLANCE SHUTDOWN UNTIL OCTOBER 8, 1980.
80-18	10/08/80	F	2.2	B	1				GENERATOR TAKEN OFF-LINE TO INVESTIGATE PROBLEMS WITH #2 STOP VALVE.
80-19	10/10/80	F	30.5	H	3				TURBINE TRIP OCCURRED WHILE PERSONNEL WERE INVESTIGATING PROBLEMS WITH BEARING VIBRATION.
80-20	10/18/80	F	.0	F	5				REACTOR AND ELECTRICAL POWER WERE REDUCED DUE TO LOW HELIUM INVENTORY. GENERATOR REMAINED ON-LINE.
80-21	10/22/80	F	1.9	H	3				TURBINE TRIP AND SCRAM DUE TO SPURIOUS LOOP SHUTDOWN DURING ULTRASONIC TESTING.
80-22	10/23/80	F	134.9	A	1				TURBINE TAKEN OFF-LINE AND REACTOR POWER REDUCED IN ORDER TO REPAIR A HYDRAULIC OIL LEAK.

SUMMARY

FORT ST. VRAIN OPERATED WITH NUMEROUS OUTAGES AND REDUCTIONS DURING OCTOBER LISTED IN DETAIL ABOVE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

NRC POWER RESTRICTIONS: NONE

GINNA

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	New York/Wayne 15 miles NE of Rochester
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	1520
5.	Design Electrical Rating (MWE Net)	470
6.	Date of Initial Criticality	November 8, 1969
7.	Date of Commercial Operation	July 1970
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Lake Ontario

Utility & Contractor Information

10.	Licensee	Rochester Gas & Electric Corporation
11.	Corporate Address	89 East Avenue Rochester, New York
12.	Corporate Contact	John Maier, Vice President Electric and Steam Production
13.	Architect/Engineer	Gilbert
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Section Chief	H. B. KISTER
19.	IE Resident Inspector	D. Johnson (315) 524-6935 FTS: 264-3311
20.	Licensing Project Manager	R. P. Snaider
21.	Docket Number	50-244
22.	License Number/Date of Issue	DPR-18, 9/19/69
23.	Public Document Room	Rochester Public Library Business and Social Science Division 115 South Avenue Rochester, New York 14604

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

```
     I N S P E C T I O N   S T A T U S

     OTHER ITEMS OF INTEREST

+    SYSTEM AND COMPONENT PROBLEMS:

+    STEAM GENERATOR INSPECTIONS AND REPAIRS ARE
     BEING PERFORMED AS A RESULT OF EDDY CURRENT
     TESTING THAT INDICATED INTERGRANULAR ATTACK
     IN THE TUBE CREVICE AREA.

     FACILITY ITEMS (PLANS AND PROCEDURES):

+    THE FACILITY WAS IN COLD SHUTDOWN ON
     NOVEMBER 1, 1980.  THE OUTAGE IS PLANNED
     FOR APPROXIMATELY 32 DAYS AND THE UNIT IS
     TENTATIVELY SCHEDULED FOR RETURN TO POWER
     ON OR ABOUT DECEMBER 1, 1980.

+    MANAGERIAL ITEMS:

+    MR. JOHN MAIER HAS REPLACED MR. LEON D. WHITE, JR.
     AS VICE PRESIDENT OF ELECTRIC AND STEAM PRODUCTION,
     EFFECTIVE AS OF OCTOBER 1, 1980.

     PLANT STATUS:

+    FULL POWER OPERATION DURING THE MONTH OF OCTOBER.

+ LAST IE SITE INSPECTION DATE:  10/31/80

+         INSPECTION REPORT NO:  50-244/80-11

     SUMMARY

     INSPECTION SUMMARY

+    50-244/80-12 - SEP 15-17: ROUTINE, UNANNOUNCED INSPECTION BY
     ONE REGION-BASED INSPECTOR (22 HRS) OF THE LICENSED OPERATOR
     REQUALIFICATION TRAINING PROGRAM, THE GENERAL EMPLOYEE TRAIN-
     ING PROGRAM, THE NON-LICENSED PERSONNEL TRAINING PROGRAM AND
     AN INDEPENDENT PLANT TOUR.  NO NONCOMPLIANCES WERE IDENTIFIED.
```

REPORTS RECEIVED FROM LICENSEE

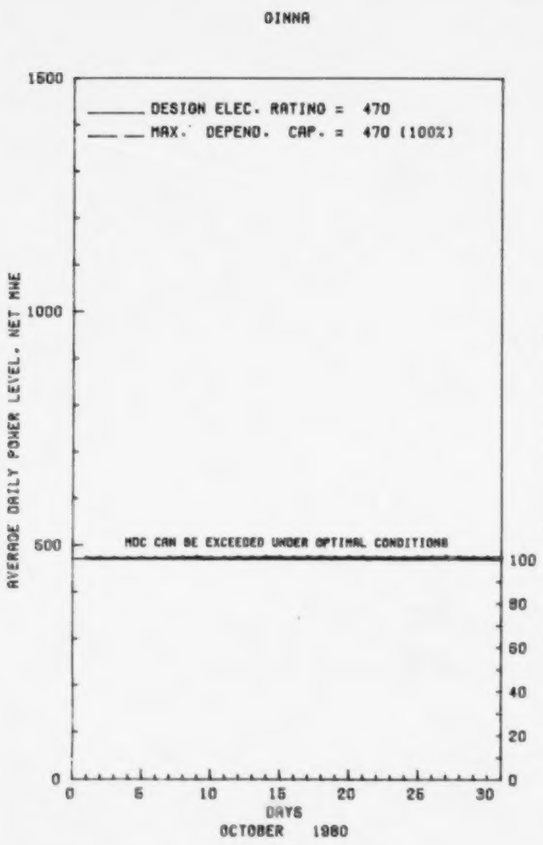
```
==============================================================================
NUMBER    DATE OF    DATE OF    SUBJECT
           EVENT     REPORT
==============================================================================
NONE
==============================================================================
```

REVIEWED BY OMPA LOEB: PE Robe **DATE:** 12/10/80

REVIEWED BY NRR DL: **DATE:** 12/11/80

GINNA

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-244
2. Utility Contact: A.E. MCNAMARA (315) 524-4446
3. Licensed Thermal Power (MWt): 1520
4. Nameplate Rating (Gross MWe): 608 x 0.85 = 517
5. Design Electrical Rating (Net MWe): 470
6. Maximum Dependable Capacity (Gross MWe): 490
7. Maximum Dependable Capacity (Net MWe): 470

CAPACITY FACTORS CALCULATED WITH A WEIGHTED AVERAGE

8. If Changes Occur in Capacity Ratings Above Since Last Report. Give Reasons:
NONE
9. Power Le... To Which Restricted. If Any (Net MWe): NONE
10. Reasons for Restrictions. If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	745.0	7,320.0	95,832.0
12. Hours Reactor Was Critical	745.0	6,019.0	73,698.8
13. Reactor Reserve Shutdown Hours	.0	.0	1,621.8
14. Hours Generator On-Line	745.0	5,980.3	71,937.1
15. Unit Reserve Shutdown Hours	.0	.0	8.5
16. Gross Thermal Energy (MWH)	1,124,472	8,890,776	97,166,578
17. Gross Electrical Energy (MWH)	370,035	2,911,829	31,597,620
18. Net Electrical Energy (MWH)	352,101	,768,385	29,929,933
19. Unit Service Factor	100.0	81.7	75.1
20. Unit Availability Factor	100.0	81.7	75.1
*21. Unit Capacity Factor (MDC Net)	100.6	80.5	68.7
22. Unit Capacity Factor (DER Net)	100.6	80.5	68.7
23. Unit Forced Outage Rate	.0	.0	9.2

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NOVEMBER 1, 1980. (STEAM GENERATOR MAINTENANCE)
25. If Shut Down At End of Report Period. Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	NONE								

SUMMARY

GINNA OPERATED AT NEAR FULL POWER THROUGHOUT OCTOBER, WITH NO REPORTABLE OUTAGES OR REDUCTIONS.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

HADDAM NECK

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Connecticut/Middlesex 13 miles E of Meriden
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	1825
5.	Design Electrical Rating (MWE Net)	580
6.	Date of Initial Criticality	July 24, 1967
7.	Date of Commercial Operation	January 1, 1968
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Connecticut River

Utility & Contractor Information

10.	Licensee	Connecticut Yankee Atomic Power Co.
11.	Corporate Address	P. O. Box 270 Hartford, Connecticut
12.	Corporate Contact	D. Switzer, President
13.	Architect/Engineer	Stone & Webster
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Section Chief	R. Keimig
19.	IE Resident Inspector	T. H. Smith (203) 267-2571,2
20.	Licensing Project Manager	R. Caruso (FTS) 244-2000
21.	Docket Number	50-213
22.	License Number/Date of Issue	DPR-61, 12/27/74
23.	Public Document Room	Russell Library 119 Broad Street Middletown, Connecticut 06457

INSPECTION STATUS

ENFORCEMENT STATUS

THREE PERSONNEL WERE EXEMPTED FROM TAKING 1979 ANNUAL EXAM VICE THE MAXIMUM OF TWO.
(8015 DEFICIENCY)

NO EVIDENCE THAT ALL OPERATORS REVIEWED ALL EOP'S IN 1979.
(8015 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS:

OPERATING AT 100% POWER.

◆ LAST IE SITE INSPECTION DATE: 10/6-10/80

◆ INSPECTION REPORT NO: 50-213/80-19

SUMMARY

INSPECTION SUMMARY

◆ 50-213/80-17 - SEP 11: MANAGEMENT MEETING HELD AT NRC'S REQUEST TO DISCUSS THE REGULATORY PERFORMANCE OF THE LICENSEE AS CONCLUDED IN THE SYSTEMATIC ASSESSMENT OF LICENSEE PERFORMANCE (SALP) PROGRAM. A SUMMARY OF THE LICENSEE PERFORMANCE EVALUATION WAS PRESENTED. AREAS OF CONCERN WERE DISCUSSED WITH CORPORATE MANAGEMENT. PERFORMANCE WAS CONSIDERED TO BE ACCEPTABLE.

REPORTS RECEIVED FROM LICENSEE

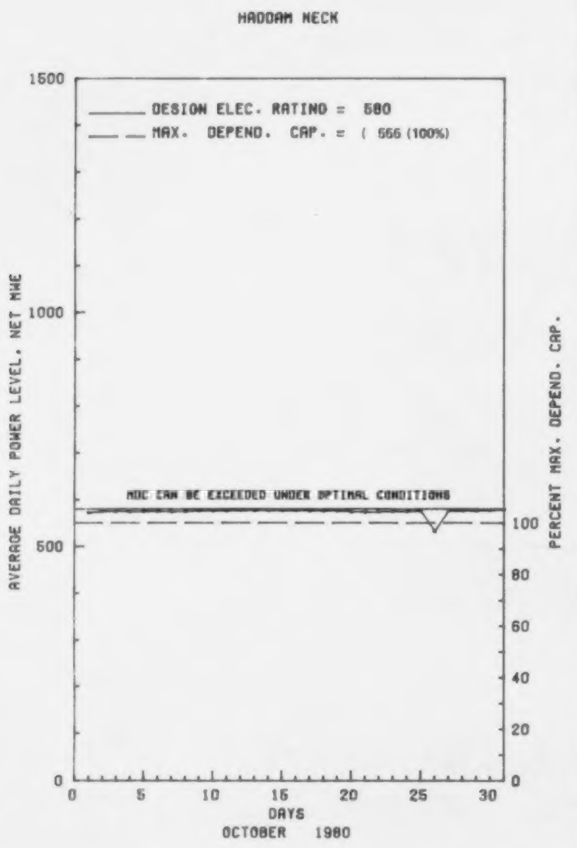
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-004E	05/28/80	05/28/80	ANOMALOUS RADIOACTIVITY OBSERVED IN RIVER.
80-005E	05/28/80	06/06/80	GAS RELEASE IN EXCESS OF TECH SPEC LIMITS.
80-006E	06/22/80	07/01/80	CURIE CONTENT IN RWST IN EXCESS OF TECH SPEC LIMIT.
80-010/ 01T	06/21/80	07/04/80	MISSING SEISMIC SUPPORTS ON CATEGORY I PIPING SYSTEMS.
80-011/ 03L	08/01/80	08/13/80	MISSED SURVEILLANCE TEST ON CONTAINMENT HATCH.
80-012/ 03L	07/17/80	08/11/80	CONTAINED LEAKAGE RATE TECH SPEC EXCEEDED WHEN THE #2 RCP SEAL BYPASS VALVE STEM WAS BROKEN.

REVIEWED BY OMPA LOEB: PE Robe **DATE:** 12/10/80

REVIEWED BY NRR DL: [illegible] **DATE:** [illegible]

HADDAM NECK

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-213
2. Utility Contact: DON ANDERSON (203) 267-2556 X274
3. Licensed Thermal Power (MWt): 1825
4. Nameplate Rating (Gross MWe): 667 x 0.9 = 600
5. Design Electrical Rating (Net MWe): 580
6. Maximum Dependable Capacity (Gross MWe): 577
7. Maximum Dependable Capacity (Net MWe): 555
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
MDC INCREASED FROM 550 TO 555; DEMONSTRATED PERFORMANCE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

CAPACITY FACTORS CALCULATED WITH A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	745.0	7,320.0	112,512.0
12. Hours Reactor Was Critical	745.0	5,323.9	96,520.6
13. Reactor Reserve Shutdown Hours	.0	27.0	1,179.6
14. Hours Generator On-Line	745.0	5,167.9	92,108.1
15. Unit Reserve Shutdown Hours	.0	.0	369.9
16. Gross Thermal Energy (MWH)	1,355,370	8,897,036	159,038,185
17. Gross Electrical Energy (MWH)	447,699	2,914,575	52,269,041
18. Net Electrical Energy (MWH)	427,876	2,767,344	49,722,107
19. Unit Service Factor	100.0	70.6	81.9
20. Unit Availability Factor	100.0	70.6	82.2
21. Unit Capacity Factor (MDC Net)	103.5	68.1	81.7
22. Unit Capacity Factor (DER Net)	99.2	65.2	75.4
23. Unit Forced Outage Rate	.0	.6	6.9

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	NONE								

SUMMARY

HADDAM NECK (CONNECTICUT YANKEE) OPERATED AT ABOVE FULL POWER FOR MOST OF OCTOBER, WITH NO REPORTABLE OUTAGES OR REDUCTIONS.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

NRC POWER RESTRICTIONS: NONE

HATCH 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Georgia/Appling 11 miles N of Baxley
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2436
5.	Design Electrical Rating (MWE Net)	777
6.	Date of Initial Criticality	September 12, 1974
7.	Date of Commercial Operation	December 31, 1975
8.	Condenser Cooling Method	Once Through With Forced Draft Cooling Towers
9.	Condenser Cooling Water Source	Altamaha River

Utility & Contractor Information

10.	Licensee	Georgia Power Company
11.	Corporate Address	270 Peachtree Street, NE Atlanta, Georgia 30303
12.	Corporate Contact	J. H. Miller Executive Vice President
13.	Architect/Engineer	Southern Services, Inc.
14.	Nuclear Steam Supply System	GE
15.	Constructor	Georgia Power Company

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. Lewis
18.	IE Section Chief	H. C. Dance
19.	IE Resident Inspector	R. F. Rogers (912) 367-9881
20.	Licensing Project Manager	D. Verrelli
21.	Docket Number	50-321
22.	License Number/Date of Issue	DPR-57, 8/6/74
23.	Public Document Room	Appling County Public Library Parker Street Baxley, Georgia 31513

INSPECTION STATUS

ENFORCEMENT STATUS

AS REQUIRED BY TECHNICAL SPECIFICATION 6.8.1, PROCEDURES CONTROLLING THE OPERATION OF SAFETY RELATED SYSTEMS ARE REQUIRED TO BE IMPLEMENTED. PLANT PROCEDURE HNP-1-1118, RHR - STANDBY CONDITION, REQUIRES MINIMUM FLOW MANUAL ISOLATION VALVES TO BE OPEN. CONTRARY TO THE ABOVE, AND AS REPORTED BY THE LICENSEE, MINIMUM FLOW MANUAL ISOLATION VALVES WERE FOUND IMPROPERLY SHUT ON "B" AND "D" RHR PUMPS ON JUNE 22, 1980.
(8031 INFRACTION)

AS REQUIRED BY 10CFR50, APPENDIX B, CRITERION XVI, AND FSAR SECTION 17.2.2.2 ITEM 1, AND IMPLEMENTED BY HATCH QUALITY ASSURANCE MANUAL, SECTION 16, PARAGRAPH 16.1, MEASURES SHALL BE ESTABLISHED TO ASSURE THAT CONDITIONS ADVERSE TO QUALITY ARE PROMPTLY IDENTIFIED AND CORRECTED. CONTRARY TO THE ABOVE, THE LICENSEE FAILED TO CORRECT SEISMIC DEFICIENCIES IN SAFETY RELATED RHR SERVICE WATER AND PLANT SERVICE WATER SYSTEMS IDENTIFIED BY THE ARCHITECT ENGINEER ON JANUARY 2, 1979 UNTIL FEBRUARY 1980.
(8031 INFRACTION)

THE FIRE DOORS BETWEEN THE UNIT 1 RADWASTE BUILDING AND UNIT 1 EAST CABLEWAY AND THE ROLLING FIRE DOOR SEPARATING DIESEL GENERATOR ROOM 1A FROM OTHER DIESEL GENERATOR ROOMS WERE BLOCKED OPEN WITHOUT A FIRE WATCH WHICH IS NOT PERMITTED BY TECH SPEC SECTION 3.13.6.
(8036 INFRACTION)

AN INDEPENDENT FIRE PROTECTION AND LOSS PREVENTION INSPECTION AND AUDIT WAS NOT CONDUCTED WITHIN THE PAST 12 MONTHS AS REQUIRED BY TECHNICAL SPECIFICATION SECTION 6.5.2.8.I.
(8036 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

PLANT IS OPERATING AT FULL POWER.

+ LAST IE SITE INSPECTION DATE: SEPTEMBER 24-25, 1980

+ INSPECTION REPORT NO: 50-321/80-40

REPORTS RECEIVED FPOM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-096/ 03L-0	09/02/80	09/23/80	DRYWELL-TORUS TEMPERATURE RECORDER INOPERABLE.
80-101/ 03L-0	08/19/80	09/09/80	MISSED SURVEILLANCE CHECKS SCRAM DISCHARGE VOLUME.
80-102/ 03L-0	08/21/80	09/09/80	"B" LOW PRESSURE COOLANT INJECTION INVERTER TRIPPED HIGH TEMPERATURE AMBIENT.
80-103/ 03L-0	08/22/80	09/09/80	PLANT SERVICE WATER STRAINERS FLOW RESTRICTIONS.
80-104/ 03L-0	08/26/80	09/09/80	FIVE SHUBBER HAVE SEALS LEAKING OIL.

SUMMARY

INSPECTION SUMMARY

+ INSPECTION SEPTEMBER 16-19 (80-36): THIS SPECIAL, UNANNOUNCED INSPECTION INVOLVED 13 INSPECTOR-HOURS ON SITE IN THE AREA OF FIRE PROTECTION/PREVENTION. OF THE AREA INSPECTED, TWO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED (INFRACTION - FAILURE TO CONDUCT ANNUAL AUDIT OF FIRE PROTECTION PROGRAM; AND INFRACTION - NONFUNCTIONAL FIRE DOORS); NO APPARENT DEVIATIONS WERE FOUND.

INSPECTION SEPTEMBER 16-17 (80-37): THIS SPECIAL, UNANNOUNCED MANAGEMENT INSPECTION INVOLVED 4 INSPECTOR-HOURS ONSITE IN THE AREA OF FOLLOWUP ON ITEMS OF NONCOMPLIANCE RELATIVE TO PART 21 EVALUATION AND REPORTING MATTERS. NONCOMPLIANCE 321/79-33-01 AND 366/79-37-01, "HATCH PART 21 PROCEDURES" ARE CLOSED.

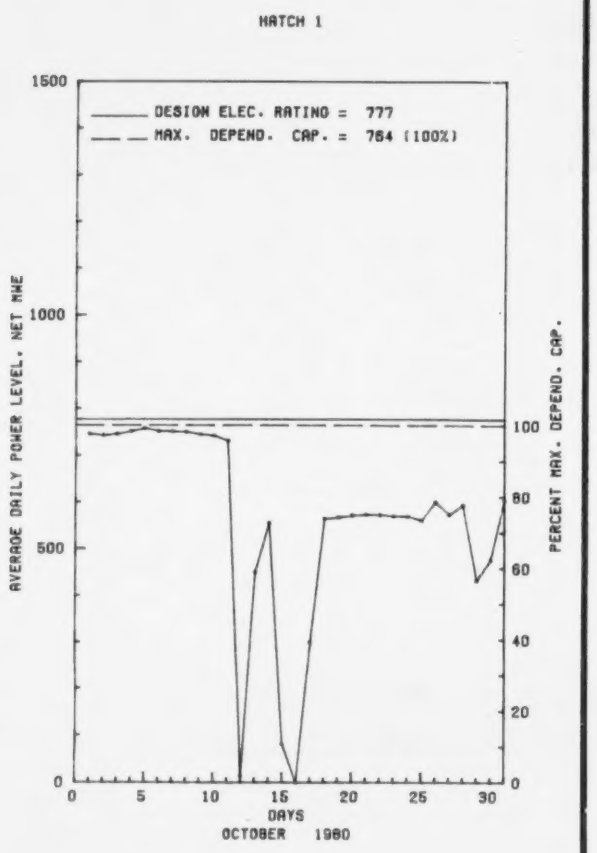
INSPECTION SEPTEMBER 22-26 (80-38): INCLUDED REVIEW OF SECURITY PLAN; SECURITY PROGRAM AUDIT; SECURITY ORGANIZATION; PHYSICAL BARRIERS (PROTECTED AND VITAL AREAS); LIGHTING; ACCESS CONTROLS (IDENTIFICATION, AUTHORIZATION AND BADGING, SEARCH, ESCORTING AND VITAL AREAS); ALARM STATIONS; DETECTION AIDS; ASSESSMENT AIDS; COMMUNICATIONS; POWER SUPPLY; LOCKS, KEYS AND COMBINATIONS. THE INSPECTION INVOLVED 17 INSPECTOR HOURS ONSITF BY 1 NRC INSPECTOR. THE INSPECTION WAS BEGUN DURING AN OFFSITE PERIOD; 2 INSPECTION HOURS WERE ACCOMPLISHED DURING OFFSHIFT PERIODS. THE LICENSEE WAS FOUND TO BE IN COMPLIANCE WITH NRC REQUIREMENTS WITHIN THE 12 AREAS EXAMINED DURING THE INSPECTION EXCEPT FOR THE FOLLOWING: PHYSICAL BARRIERS - VITAL AREA BARRIER RESISTANCE TO PENETRATION DIMINISHED BY INADEQUATE VENT COVERING.

INSPECTION SEPTEMBER 24-25 (80-40): THIS ROUTINE UNANNOUNCED INSPECTION INVOLVED 6 INSPECTOR-HOURS ONSITE IN THE AREAS OF: EXAMINATION OF CONTAINMENT LINER PENETRATION WELDS PER BULLETIN 80-08; FOLLOWUP OF TWO NONCOMPLIANCE CONCERNING APPROVAL OF THE INSERVICE INSPECTION PLAN AND CERTIFICATION OF WELDING AND NDE INSPECTORS; AND REVIEW OF ACTIONS TAKEN ON INSPECTOR FOLLOWUP ITEMS CONCERNING EVALUATION OF A LINEAR INDICATION. NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 12-10-80
REVIEWED BY NRR DL: M.B. Fairtile **DATE:** 12/11/80

HATCH 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-321
2. Utility Contact: P. R. UNDERWOOD (912) 367-7781
3. Licensed Thermal Power (MWt): 2436
4. Nameplate Rating (Gross MWe): 1000 x 0.85 = 850
5. Design Electrical Rating (Net MWe): 777
6. Maximum Dependable Capacity (Gross MWe): 797
7. Maximum Dependable Capacity (Net MWe): 764
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	745.0	7,320.0	42,384.0
12. Hours Reactor Was Critical	715.1	6,308.4	32,106.4
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	676.0	5,829.1	30,097.5
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,373,555	12,964,362	63,317,397
17. Gross Electrical Energy (MWH)	438,900	4,223,730	20,494,670
18. Net Electrical Energy (MWH)	417,932	4,024,826	19,486,759
19. Unit Service Factor	90.7	79.6	71.0
20. Unit Availability Factor	90.7	79.6	71.0
21. Unit Capacity Factor (MDC Net)	73.4	72.0	60.2
22. Unit Capacity Factor (DER Net)	72.2	70.8	59.2
23. Unit Forced Outage Rate	9.3	20.4	17.0

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
54	10/12/80	F	20.9	A	3		HA	VALVEX	RX SCRAM DUE TO TSV CLOSURE WHICH WAS CAUSED BY A POWER LOAD UNBALANCE ON MAIN GENERATOR.
55	10/12/80	F	17.1	A	5		HA	VALVEX	UNIT ON STARTUP RAMP FROM ABOVE SHUTDOWN #80-54.
56	10/12/80	F	33.5	F	5		ZZ	ZZZZZZ	MANAGEMENT LIMITATION OF 75% THERMAL POWER.
57	10/15/80	F	7.1	A	5		HA	GENERA	REDUCTION OF LOAD DUE TO GENERATOR GROUND FAULT ALARM.
58	10/15/80	F	45.5	A	1		HA	GENERA	UNIT OFF-LINE TO INVESTIGATE GROUND FAULT ALARM IN #80-57.
59	10/16/80	F	19.4	A	5		HA	GENERA	UNIT ON STARTUP RAMP FROM ABOVE SHUTDOWN, #80-58.
60	10/18/80	F	166.8	F	5		ZZ	ZZZZZZ	MANAGEMENT LIMITATION OF 75% THERMAL POWER.
61	10/24/80	F	5.8	B	5		HE	TURBIN	REDUCED LOAD TO PERFORM WEEKLY TURBINE TEST.
62	10/25/80	F	107.0	F	5		ZZ	ZZZZZZ	MANAGEMENT LIMITATION OF 75% THERMAL POWER.
63	10/29/80	F	5.4	A	5		HA	GENERA	REDUCED LOAD BECAUSE OF GENERATOR GROUND FAULT ALARM.
64	10/29/80	F	2.6	A	1		HA	GENERA	UNIT OFF-LINE TO INVESTIGATE GROUND FAULT ALARM IN #80-63.
65	10/29/80	F	12.2	A	5		HA	GENERA	UNIT ON STARTUP RAMP FROM ABOVE SHUTDOWN #80-64.
66	10/30/80	F	37.5	F	5		ZZ	ZZZZZZ	MANAGEMENT LIMITATION OF 80% THERMAL POWER.

SUMMARY

HATCH 1 OPERATED NORMALLY DURING OCTOBER, WITH NUMEROUS OUTAGES AND REDUCTIONS LISTED ABOVE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

HATCH 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1. Location (state/county) (nearest population center): Georgia/Appling, 11 miles N of Baxley
2. Electric Reliability Council: Southeastern Electric Reliability Council
3. Type of Reactor: BWR
4. Power Level (MWT): 2436
5. Design Electrical Rating (MWE Net): 784
6. Date of Initial Criticality: July 4, 1978
7. Date of Commercial Operation: September 5, 1979
8. Condenser Cooling Method: Cooling Towers
9. Condenser Cooling Water Source: Altamaha River

Utility & Contractor Information

10. Licensee: Georgia Power Company
11. Corporate Address: 270 Peach Street, NE, Atlanta, Georgia 30303
12. Corporate Contact: J. H. Miller, Executive Vice President
13. Architect/Engineer: Southern Services, Inc./Bechtel
14. Nuclear Steam Supply System: GE
15. Constructor: Georgia Power Company

Regulatory Information

16. IE Region responsible: II
17. IE Branch Chief: R. C. Lewis
18. IE Section Chief: H. C. Dance
19. IE Resident Inspector: R. F. Rogers (912) 367-9881
20. Licensing Project Manager: D. Verrelli
21. Docket Number: 50-366
22. License Number/Date of Issue: NPF-5/June 13, 1978
23. Public Document Room: Appling County Public Library, Parker Street, Baxley, Georgia 31513

INSPECTION STATUS

ENFORCEMENT STATUS

AN INDEPENDENT FIRE PROTECTION AND LOSS PREVENTION INSPECTION AND AUDIT WAS NOT CONDUCTED WITHIN THE PAST 12 MONTHS AS REQUIRED BY TECHNICAL SPECIFICATION SECTION 6.5.2.8.I.
(8036 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

- REACTOR IS IN COLD SHUTDOWN FOR REFUELING.
- LAST IE SITE INSPECTION DATE: SEPTEMBER 24-25, 1980
- INSPECTION REPORT NO: 50-366/80-40

SUMMARY

INSPECTION SUMMARY

- INSPECTION SEPTEMBER 16-19 (80-36): THIS SPECIAL, UNANNOUNCED INSPECTION INVOLVED 13 INSPECTOR-HOURS ON SITE IN THE AREA OF FIRE PROTECTION/PREVENTION. OF THE AREA INSPECTED, TWO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED (INFRACTION - FAILURE TO CONDUCT ANNUAL AUDIT OF FIRE PROTECTION PROGRAM; AND INFRACTION - NONFUNCTIONAL FIRE DOORS); NO APPARENT DEVIATIONS WERE FOUND.

INSPECTION SEPTEMBER 16-17 (80-37): THIS SPECIAL, UNANNOUNCED MANAGEMENT INSPECTION INVOLVED 4 INSPECTOR-HOURS ONSITE IN THE AREA OF FOLLOWUP ON ITEMS OF NONCOMPLIANCE RELATIVE TO PART 21 EVALUATION AND REPORTING MATTERS. NONCOMPLIANCE 321/79-33-01 AND 366/79-37-01, "HATCH PART 21 PROCEDURES" ARE CLOSED.

INSPECTION SEPTEMBER 22-26 (80-38): INCLUDED REVIEW OF SECURITY PLAN; SECURITY PROGRAM AUDIT; SECURITY ORGANIZATION; PHYSICAL BARRIERS (PROTECTED AND VITAL AREAS); LIGHTING; ACCESS CONTROLS (IDENTIFICATION, AUTHORIZATION AND BADGING, SEARCH, ESCORTING AND VITAL AREAS); ALARM STATIONS; DETECTION AIDS; ASSESSMENT AIDS; COMMUNICATIONS; POWER SUPPLY; LOCKS, KEYS AND COMBINATIONS. THE INSPECTION INVOLVED 17 INSPECTOR HOURS ONSITE BY 1 NRC INSPECTOR. THE INSPECTION WAS BEGUN DURING AN OFFSITE PERIOD; 2 INSPECTION HOURS WERE ACCOMPLISHED DURING OFFSHIFT PERIODS. THE LICENSEE WAS FOUND TO BE IN COMPLIANCE WITH NRC REQUIREMENTS WITHIN THE 12 AREAS EXAMINED DURING THE INSPECTION EXCEPT FOR THE FOLLOWING: PHYSICAL BARRIERS - VITAL AREA BARRIER RESISTANCE TO PENETRATION DIMINISHED BY INADEQUATE VENT COVERING.

INSPECTION SEPTEMBER 24-25 (80-40): THIS ROUTINE UNANNOUNCED INSPECTION INVOLVED 6 INSPECTOR-HOURS ONSITE IN THE AREAS OF: EXAMINATION OF CONTAINMENT LINER PENETRATION WELDS PER BULLETIN 80-08; FOLLOWUP OF TWO NONCOMPLIANCE CONCERNING APPROVAL OF THE INSERVICE INSPECTION PLAN AND CERTIFICATION OF WELDING AND NDE INSPECTORS; AND REVIEW OF ACTIONS TAKEN ON INSPECTOR FOLLOWUP ITEMS CONCERNING EVALUATION OF A LINEAR INDICATION. NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

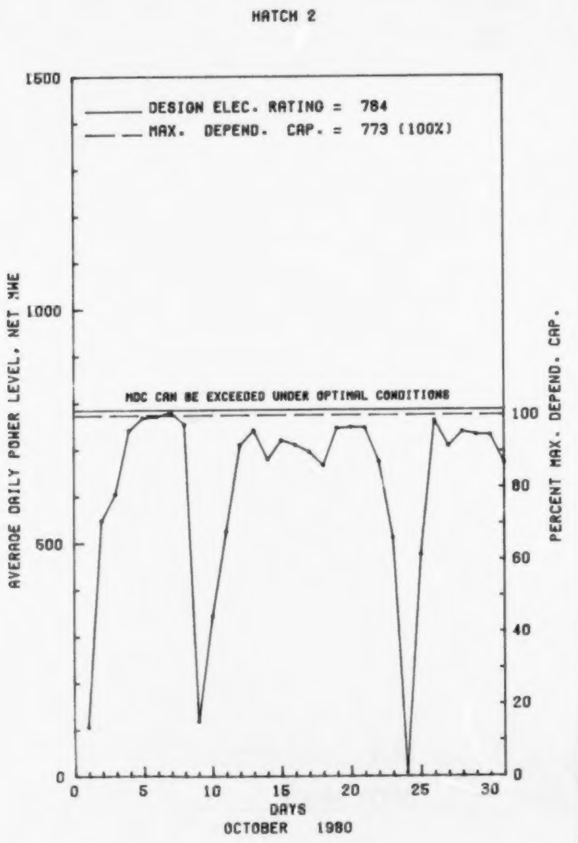
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-105/ 03L-0	08/21/80	09/18/80	LOST MOMENTARILY STANDBY LIQUID CONTROL SYSTEM.
80-125/ 03L-0	08/24/80	09/11/80	MSIV INDICATOR LIGHT INOPERABLE.
80-126/ 03L-0	08/24/80	09/09/80	PRIMARY CONTAINMENT HYDROGEN-OXYGEN ANALYZER INOPERABLE.
80-127/ 03L-0	08/30/80	09/23/80	2C DIESEL FAILED TO START DURING TESTING.
80-128/ 03L-0	09/05/80	09/25/80	REACTOR PRESSURE INSTRUMENT SET POINT DRIFT.
80-132/ 03L-0	09/10/80	09/30/80	CORE SPRAY PUMP INTERLOCK SET LOW.

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 12-10-80

REVIEWED BY NRR DL: W. B. Fairtile **DATE:** 12/11/80

HATCH 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-366
2. Utility Contact: P. R. UNDERWOOD (912) 367-7781
3. Licensed Thermal Power (MWt): 2436
4. Nameplate Rating (Gross MWe): 1000 x .85 = 850
5. Design Electrical Rating (Net MWe): 784
6. Maximum Dependable Capacity (Gross MWe): 806
7. Maximum Dependable Capacity (Net MWe): 773
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	745.0	7,320.0	10,153.0
12. Hours Reactor Was Critical	717.1	5,567.4	5,230.9
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	697.6	5,270.9	7,685.6
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,494,254	11,601,014	17,095,076
17. Gross Electrical Energy (MWH)	483,290	3,830,650	5,666,610
18. Net Electrical Energy (MWH)	461,051	3,650,398	5,407,535
19. Unit Service Factor	93.6	72.0	75.7
20. Unit Availability Factor	93.6	72.0	75.7
21. Unit Capacity Factor (MDC Net)	80.1	64.5	68.9
22. Unit Capacity Factor (DER Net)	78.9	63.6	67.9
23. Unit Forced Outage Rate	6.4	12.4	10.9

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
40	10/01/80	F	10.5	A	5		HC	HTEXCH	UNIT ON STARTUP RAMP FROM PREVIOUS SHUTDOWN #80-38.
41	10/01/80	F	11.1	A	1		HA	VALVEX	RX SCRAM DUE TO TSV CLOSURE WHICH TRIPPED TURBINE ON HWL.
42	10/01/80	F	13.4	A	5		HA	VALVEX	UNIT ON STARTUP RAMP FROM ABOVE SHUTDOWN #80-41.
43	10/03/80	F	15.5	H	5		RB	CONROD	REDUCED LOAD IN ORDER TO PERFORM A ROD PATTERN ADJUSTMENT.
44	10/08/80	F	6.3	A	5		CH	HTEXCH	REDUCTION OF LOAD BECAUSE OF PROBLEMS WITH FEEDWATER HEATERS
45	10/09/80	F	15.7	A	1		HH	PUMPXX	RX SCRAM DUE TO CONDENSER BOOSTER PUMP TRIP ON LOW SUCTION WHICH TRIPPED THE RFP'S.
46	10/09/80	F	49.8	A	5		HC	HTEXCH	UNIT ON STARTUP RAMP FROM ABOVE SHUTDOWN #80-46.
47	10/17/80	F	8.6	H	5		RB	CONROD	REDUCED LOAD IN ORDER TO PERFORM A ROD PATTERN ADJUSTMENT.
48	10/22/80	F	14.6	A	5		HC	HTEXCH	REDUCED LOAD TO INVESTIGATE POSSIBLE TUBE FAILURE IN CONDENSER.
49	10/23/80	F	5.6	A	5		HF	HTEXCH	REDUCED LOAD TO REPAIR LEAK IN WATERBOX.
50	10/24/80	F	20.6	A	3		HF	HTEXCH	UNIT OFF-LINE AND SHUTDOWN TO REPAIR LEAK IN WATERBOX.
51	10/24/80	F	23.6	A	5		HF	HTEXCH	UNIT ON STARTUP RAMP FROM ABOVE SHUTDOWN #80-50.
52	10/31/80	S	5.3	C	5		ZZ	ZZZZZZ	REDUCED LOAD TO SHUTDOWN FOR REFUEL-OUTAGE.

SUMMARY

HATCH 2 OPERATED NORMALLY DURING OCTOBER, WITH NUMEROUS OUTAGES AND REDUCTIONS LISTED ABOVE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

HUMBOLDT BAY

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	California/Humboldt 4 miles S of Eureka
2.	Electric Reliability Council	Western Systems Coordinating Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	220
5.	Design Electrical Rating (MWE Net)	65
6.	Date of Initial Criticality	February 16, 1963
7.	Date of Commercial Operation	August 1963
8.	Condenser Cooling Method	Circulating sea water
9.	Condenser Cooling Water Source	Pacific Ocean (Humboldt Bay)

Utility & Contractor Information

10.	Licensee	Pacific Gas & Electric Co.
11.	Corporate Address	77 Beale Street San Francisco, California 94106
12.	Corporate Contact	P. A. Crane, Jr. Asst. General Counsel
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	GE
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	V
17.	IE Branch Chief	J. L. Crews
18.	IE Section Chief	D. M. Sternberg
19.	IE Resident Insp[illegible]	J. D. Carlson
20.	Licensing Project Manager	V. L. Rooney
21.	Docket Number	50-133
22.	License Number/Date of Issue	DPR-7, 8/28/62
23.	Public Document Room	Humboldt County Library 636 F Street Eureka, California 95501

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

THE PLANT HAS BEEN SHUT DOWN SINCE JULY 1976.

ITEMS REMAINING TO BE COMPLETED DURING THE OUTAGE HAVE BEEN DEFERRED PENDING RESOLUTION OF THE SEISMIC ISSUES AT THE PLANT SITE. A DATE FOR RETURN TO OPERATIONS HAS NOT BEEN ESTABLISHED.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

PLANT IS SHUTDOWN.

LAST IE SITE INSPECTION DATE: 05/20-22/80

INSPECTION REPORT NO: 50-133/80-02

SUMMARY

INSPECTION SUMMARY

- NO INSPECTIONS CONDUCTED IN OCTOBER 1980.

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-05/ 03L-0	09-26-80	10-26-80	FAILURE OF CROSS-TIE LINE IN SCREEN WASH SYSTEM.
80-06/ 03L-0	09-26-80	10-26-80	FAILURE OF SCREEN WASH PUMP DISCHARGE VALVE.

LICENSE STATUS

Order issued 5/21/76 requiring Utility to prove seismic qualification before resuming operation of unit after outage of 7/2/76.

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 12-10-80

REVIEWED BY NRR DL: V Rooney **DATE:** 12/10/80

HUMBOLDT BAY

AVERAGE DAILY POWER LEVEL (MWe)

OPERATING STATUS

NO DECISION ON FUTURE OPERATION
SHUTDOWN INDEFINITELY

1. Docket Number: 50-133
2. Utility Data Prepared By: B. C. Getty (707) 443-2787
3. Licensed Thermal Power (MWt): 220
4. Nameplate Rating (Gross MWe): 77 X 0.85 = 65
5. Design Electrical Rating (Net MWe): 65
6. Maximum Dependable Capacity (Gross MWe): 0
7. Maximum Dependable Capacity (Net MWe): 0
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons
None
9. Power Level To Which Restricted, If Any (Net MWe): None
10. Reasons For Restrictions, If Any:

This ... ive

11. Hours In Reporting Period
12. Number Of Hours Reactor Was Cri...
13. Reactor Reserve Shutdow...
14. Hours Generator ...
15. Unit R...
1... rt)
... erated (MWH)
... Net ... gy Generated (MWH)
19. Unit S... ce Factor
20. Unit Availability Factor
21. Unit Capacity Factor (Using MDC Net) 1/
22. Unit Capacity Factor (Using DER Net) 1/
23. Unit Forced Outage Rate
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
None
25. If Shut Down At End Of Report Period, Estimated Date of Startup: UNKNOWN

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
SEE SUMMARY BELOW									

SUMMARY

THE UNIT WAS SHUTDOWN ON 7/2/76 FOR SEISMIC MODIFICATIONS. THE UNIT REMAINS IN THE COLD SHUTDOWN CONDITION.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

INDIAN POINT 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	New York/Westchester 25 miles N of New York City
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2758
5.	Design Electrical Rating (MWE Net)	873
6.	Date of Initial Criticality	May 22, 1973
7.	Date of Commercial Operation	August 1973
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Hudson River

Utility & Contractor Information

10.	Licensee	Consolidated Edison Company of New York, Inc.
11.	Corporate Address	4 Irving Place New York, New York
12.	Corporate Contact	J. O'Toole, Asst. Vice President
13.	Architect/Engineer	UE&C
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	J. A. Jones/UE&C

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Senior Inspectc	H. B. KISTER
19.	IE Resident Inspector	T. Rebelowski (914-739-9360) FTS: 264-3311
20.	Licensing Project Manager	L. Olshan (X27144)
21.	Docket Number	50-247
22.	License Number/Date of Issue	DPR 26/9-28-73
23.	Public Document Room	White Plains Public Library 100 Martine Avenue White Plains, New York 10601

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO TECHNICAL SPECIFICATION 6.8.1; ANSI N18.7-1972, 5.3.6 AND 5.3.4 - PROCEDURES WERE INADEQUATELY MAINTAINED TO REFLECT THE ACTUAL TEST CONDITIONS.
(8013 DEFICIENCY)

CONTRARY TO TECHNICAL SPECIFICATION 6.8.1; ANSI N18.7-1972, 5.1.2; ADMINISTRATIVE DIRECTIVE TAD-0, PARAGRAPH 2.1 - CALIBRATION ACTIVITIES WERE PERFORMED WITHOUT FOLLOWING STEP-WISE INSTRUCTIONS IN THE PROCEDURES.
(8013 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

- SYSTEMS AND COMPONENT PROBLEMS:
- DUE TO EXCESSIVE DEGRADATION FIVE (5) CONTAINMENT FAN COOLER COILS AND ASSOCIATED HARDWARE WILL BE REPLACED DURING THE REFUELING OUTAGE.
- FACILITY ITEMS (PLANS AND PROCEDURES):
- THE LICENSEE HAS BEGUN THEIR REFUELING OUTAGE PREVIOUSLY SCHEDULED TO BEGIN ON 12/15/80. THE DECISION TO BEGIN THE OUTAGE EARLY WAS PROMPTED BY EXCESSIVE SERVICE WATER LEAKAGE ASSOCIATED WITH THE CONTAINMENT FAN COOLER UNITS.

PLANT STATUS:

- COLD SHUTDOWN.

- LAST IE SITE INSPECTION DATE: 10/22/80
- INSPECTION REPORT NO: 50-247/80-19

SUMMARY

INSPECTION SUMMARY

- 50-247/80-09 - JUN 29 - AUG 23: ROUTINE, ONSITE, REGULAR AND BACKSHIFT INSPECTIONS BY TWO RESIDENT INSPECTORS (42 HRS) OF PLANT OPERATIONS INCLUDING SHIFT LOGS AND RECORDS; OBSERVATION OF PHYSICAL SECURITY; REVIEW OF MONTHLY AND PERIODIC REPORTS; PLANT TOURS; VERIFICATION OF IMPLEMENTATION OF THE RADIOACTIVE WASTE SYSTEMS CONTROLS; MATERIAL STATUS REPORT; SPECIAL MANAGEMENT INSPECTION; SURVEILLANCE AND OBSERVATIONS; WESPAC DEMONSTRATION; INDUSTRIAL ACCIDENT AND PLANT TRIP AND SAFETY SYSTEM CHALLENGES. NO NONCOMPLIANCES WERE IDENTIFIED.
- 50-247/80-13 - AUG 18-22: ROUTINE, ONSITE, UNANNOUNCED INSPECTION BY TWO REGION-BASED INSPECTORS (60 HRS) OF LICENSEE ACTION ON PREVIOUS INSPECTION FINDINGS; ADMINISTRATIVE CONTROL OF SAFETY RELATED CALIBRATIONS; SURVEILLANCE CALIBRATION OF COMPONENTS AND EQUIPMENT REQUIRED BY TECHNICAL SPECIFICATIONS; CALIBRATION OF COMPONENTS AND EQUIPMENT ASSOCIATED WITH SAFETY RELATED SYSTEMS AND/OR FUNCTIONS; WITNESSING OF CALIBRATIONS; CALIBRATION AND CONTROL OF MEASURING AND TEST EQUIPMENT; TECHNICIAN QUALIFICATIONS; AND CONTROL ROOM AND FACILITY TOURS. TWO NONCOMPLIANCES WERE IDENTIFIED: FAILURE TO FOLLOW WRITTEN PROCEDURES; FAILURE TO PROPERLY REVIEW PROCEDURES AND DATA FOR ADEQUACY.

REPORTS RECEIVED FROM LICENSEE

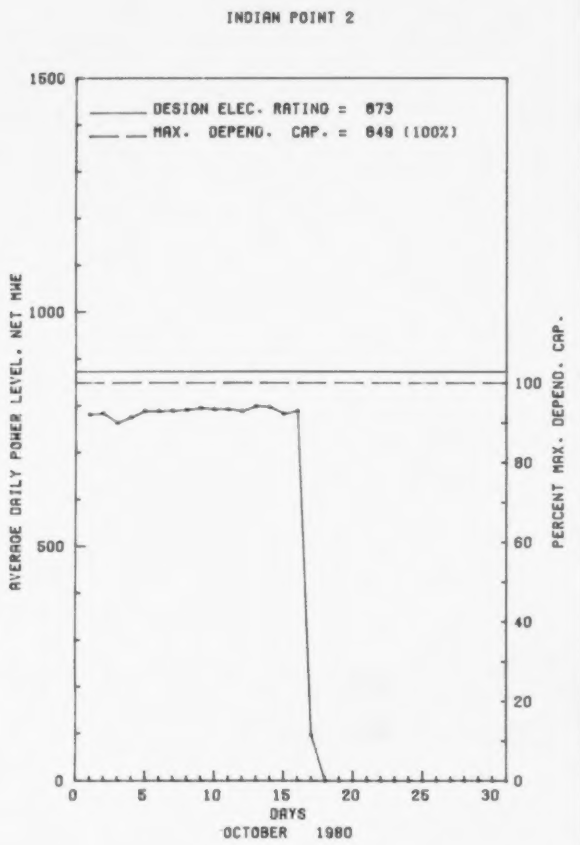
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-010/04X	09/10/80	09/19/80	ENVIRONMENTAL TECH SPEC EXCEEDED IN AREA OF PLANT SAMPLE ISOTOPE CONTENT
80-011/01T	09/24/80	10/08/80	CONTAINMENT SPRAY PUMPS HAND SWITCHES FOUND IN PULL-TO-LOCK DURING PLANT OPERATION

REVIEWED BY OMPA LOEB: ________ **DATE:** 12/10/80

REVIEWED BY NRR DL: ________ **DATE:** 12/10/80

INDIAN POINT 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-247
2. Utility Contact: E. EICH (914) 694-6000 @ I.P.
3. Licensed Thermal Power (MWt): 2758
4. Nameplate Rating (Gross MWe): 1126 x 0.9 = 1013
5. Design Electrical Rating (Net MWe): 873
6. Maximum Dependable Capacity (Gross MWe): 885
7. Maximum Dependable Capacity (Net MWe): 849
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): 824
10. Reasons for Restrictions, If Any:
REMOVAL OF #3 DISC. (GEN. END) ON #23 TURBINE ROTOR.

ITEM 21 CALCULATED BY USING A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	745.0	7,320.0	55,561.0*
12. Hours Reactor Was Critical	394.5	5,869.4	38,163.9
13. Reactor Reserve Shutdown Hours	347.7	1,231.2	1,472.6
14. Hours Generator On-Line	388.3	5,691.2	37,080.4
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,066,206	14,975,902	95,808,428
17. Gross Electrical Energy (MWH)	318,370	4,486,960	29,620,046
18. Net Electrical Energy (MWH)	299,391	4,273,726	28,238,826
19. Unit Service Factor	52.1	77.7	66.7
20. Unit Availability Factor	52.1	77.7	66.7
21. Unit Capacity Factor (MDC Net)	47.3	68.3	59.1
22. Unit Capacity Factor (DER Net)	46.0	66.9	58.2
23. Unit Forced Outage Rate	47.9	13.8	9.0

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 05/31/81

*COMPUTED SINCE 7/1/74, THE DATE OF COMPLETION OF A 100 DAY – 100% POWER OPERATION TEST.

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
14	10/17/80	F	356.7	A	3		HA	INSTRU	LOCAL TURBINE LOAD LIMIT MOVED IN DECREASING DIRECTION CAUSING SYSTEM TRANSIENT AND RESULTANT REACTOR TRIP VIA HIGH PRESSURIZER PRESSURE SIGNAL. UNIT BROUGHT TO COLD SHUTDOWN ON 10-22-80 TO ASSESS EQUIPMENT DAMAGE FROM SUBMERGENCE RESULTING FROM FCU LEAKAGE. ALSO CYCLE 4/5 REFUELING TO BE PERFORMED.

SUMMARY

INDIAN POINT 2 OPERATED NORMALLY THRU OCTOBER 17TH, WHEN AN OUTAGE OCCURRED DUE TO EQUIPMENT FAILURE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

INDIAN POINT 3

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	New York/Westchester 25 miles N of New York City
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	3025
5.	Design Electrical Rating (MWE Net)	965
6.	Date of Initial Criticality	April 6, 1976
7.	Date of Commercial Operation	August 30, 1976
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Hudson River

Utility & Contractor Information

10.	Licensee	Power Authority of the State of New York, Inc.
11.	Corporate Address	10 Columbus Circle New York, New York
12.	Corporate Contact	J. P. Bayne - Senior Vice President
13.	Architect/Engineer	UE&C
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	WEDCO

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Section Chief	H. B. KISTER
19.	IE Resident Inspector	T. Rebelowski (914-739-9360) FTS: 264-3311 - Operator
20.	Licensing Project Manager	L. Olshan x27144
21.	Docket Number	50-286
22.	License Number/Date of Issue	DPR-64, 12/12/75
23.	Public Document Room	White Plains Public Library 100 Martine Avenue White Plains, New York 10601

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

FACILITY ITEMS (PLANS AND PROCEDURES):

+ THE LICENSEE PRESENTLY PLANS TO RETURN THE PLANT TO POWER OPERATION BY MID-NOVEMBER FOLLOWING A SHORT OUTAGE FOR TURBINE AND STEAM GENERATOR INSPECTIONS.

PLANT STATUS:

+ COLD SHUTDOWN.

+ LAST IE SITE INSPECTION DATE: 10/26-31/80

+ INSPECTION REPORT NO: 50-286/80-17

SUMMARY

INSPECTION SUMMARY

+ 50-286/80-08 - JUN 8 - AUG 2: ROUTINE, ONSITE, REGULAR AND BACK-SHIFT INSPECTIONS BY TWO RESIDENT INSPECTORS (81 HRS) AND TWO REGION-BASED INSPECTORS (14 HRS) OF PLANT OPERATIONS INCLUDING SHIFT LOGS AND RECORDS; LICENSEE ACTION ON PREVIOUS INSPECTION FINDINGS; OBSERVATION OF PHYSICAL SECURITY; REVIEW OF MONTHLY AND PERIODIC REPORTS; EMERGENCY SAFEGUARDS SYSTEM READINESS AUDIT; VERIFICATION OF LICENSEE APPLICATION SUBMITTALS FOR LICENSEE TRAINING STAFF; REVIEW AND VERIFICATION OF INTERIM MEASURES, CONFIRMATORY ORDERS - 120 DAY REQUIREMENTS; COMMISSIONER'S MEETING ON RISK ASSESSMENT; INITIAL IMPLEMENTATION OF STRIKE PLANS; ENVIRONMENTAL QUALIFICATION OF ELECTRICAL EQUIPMENT; PLANT TRIPS AND SAFETY CHALLENGES; FOLLOWUP ON SIGNIFICANT EVENTS; WORKER OVEREXPOSURE AND I&E BULLETIN REVIEW. NO NON-COMPLIANCES WERE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

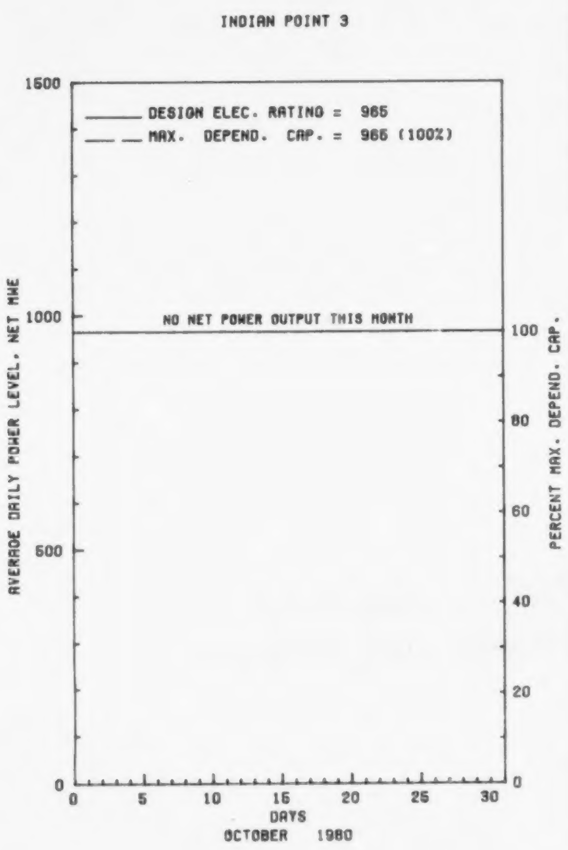
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE			

REVIEWED BY OMPA LOEB: RLBobe DATE: 12/10/80

REVIEWED BY NRR DL: DATE: 12/10/80

INDIAN POINT 3

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-286
2. Utility Contact: C. Connell (914) 739-8200 x242
3. Licensed Thermal Power (MWt): 3025
4. Nameplate Rating (Gross MWe): 1126 x 0.9 = 1013
5. Design Electrical Rating (Net MWe): 965
6. Maximum Dependable Capacity (Gross MWe): 1000
7. Maximum Dependable Capacity (Net MWe): 965
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): 917
10. Reasons for Restrictions, If Any:
REMOVED SOME TURBINE BLADING

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	745.0	7,320.0	36,577.0
12. Hours Reactor Was Critical	.5	4,856.3	26,441.2
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	.0	4,386.7	25,419.5
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	10,800,817	66,905,284
17. Gross Electrical Energy (MWH)	0	3,017,310	21,382,411
18. Net Electrical Energy (MWH)	0	2,871,081	20,514,518
19. Unit Service Factor	.0	59.9	69.5
20. Unit Availability Factor	.0	59.9	69.5
21. Unit Capacity Factor (MDC Net)	.0	40.6	58.1
22. Unit Capacity Factor (DER Net)	.0	40.6	58.1
23. Unit Forced Outage Rate	.0	20.1	7.4

24. Shutdowns Scheduled Over Next 6 Months (Type,Date,and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 11/01/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
29	09/30/80	S	745.0	B	4		ZZ	ZZZZZZ	TURBINE OUTAGE (CONTINUED).

SUMMARY

INDIAN POINT 3 REMAINED OFF-LINE FOR OCTOBER DUE TO A CONTINUING TURBINE OUTAGE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

KEWAUNEE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Wisconsin/Kewaunee 27 miles E of Green Bay
2.	Electric Reliability Council	Mid-America Interpool Network
3.	Type of Reactor	PWR
4.	Power Level (MWT)	1650
5.	Design Electrical Rating (MWE Net)	535
6.	Date of Initial Criticality	March 7, 1974
7.	Date of Commercial Operation	June 1974
8.	Condenser Cooling Method	Once through condenser and return to Lake Michigan
9.	Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10.	Licensee	Wisconsin Public Service Corp.
11.	Corporate Address	P. O. Box 1200 Green Bay, Wisconsin 54305
12.	Corporate Contact	E. R. Mathews Power Supply & Eng.
13.	Architect/Engineer	Fluor Power Services Corp.
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Pioneer Services & Eng. Co.

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	R. F. Warnick
19.	IE Resident Inspector	R. L. Nelson
20.	Licensing Project Manager	R. B. Licciardo X28362
21.	Docket Number	50-305
22.	License Number/Date of Issue	DPR-43 issued 12/21/73
23.	Public Document Room	Kewaunee Public Library 822 Juneau Street Kewaunee, Wisconsin 54216

INSPECTION STATUS

ENFORCEMENT STATUS

10 CFR PART 50, SECTION 50.55A(G) IN PART REQUIRES HOLDERS OF AN OPERATING LICENSE TO IMPLEMENT ASME CODE SECTION XI REQUIREMENTS TO TEST SELECTED PUMPS AND VALVES. CONTRARY TO THE ABOVE, THE LICENSEE FAILED TO FULLY IMPLEMENT ASME CODE SECTION XI TESTING REQUIREMENTS TO INSURE THE OPERABILITY OF SAFETY RELATED EQUIPMENT IN THAT: (A) THE MONTHLY INSERVICE INSPECTION TESTS WERE NOT CONDUCTED ON HIGH HEAD SAFETY INJECTION PUMPS, RESIDUAL HEAT REMOVAL PUMPS, AUXILIARY FEEDWATER PUMPS (MOTOR DRIVEN), COMPONENT COOLING WATER PUMPS, AND CONTAINMENT SPRAY PUMPS DURING THE MAY-JUNE REFUELING OUTAGE OR WITHIN ONE WEEK AFTER THE PLANT WAS RETURNED TO NORMAL OPERATION ON JUNE 24, 1980; (B) THE QUARTERLY INSERVICE INSPECTION TESTS OF CATEGORY A AND B VALVES ASSOCIATED WITH SURVEILLANCE PROCEDURES 098, 099, 104, 167-1, 167-3 AND 167-5 WERE NOT PERFORMED IN ACCORDANCE WITH IWV-3410 OF SECTION XI; (C) REFERENCE VALUES AS SPECIFIED IN TABLE IWP-3100-1 WERE NOT ESTABLISHED FOR THE TURBINE DRIVEN AUXILIARY FEEDWATER PUMP; AND (D) LIMITING VALVES FOR STROKE TIME HAVE NOT BEEN SPECIFIED ON THE INSERVICE DATA SHEET FOR THE FOLLOWING VALVES ASSOCIATED WITH SP-167-6: SI-15A & B; RHR-300A & B; RHR-400A & B; LD-6; CVC-211, 212; RBV-1, 2, 3, 4; VB-10A & B; FW-12A & B; ICS 5A & B, 6A & B; AND CI-1001A & B. (8018 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

MANAGERIAL ITEMS

♦ MANAGEMENT MEETING ON SEPTEMBER 19, 1980 BETWEEN THE WISCONSIN PUBLIC SERVICE CORPORATION PRESIDENT AND THE NRC REGION III DIRECTOR TO DISCUSS CORPORATE LEVEL INTERFACE RELATIONSHIP. (80-22)

PLANT STATUS

♦ THE PLANT OPERATED AT NEARLY FULL POWER DURING MOST OF THE MONTH.

♦LAST IE SITE INSPECTION DATE: SEPTEMBER 19, 1980

♦ INSPECTION REPORT NO: 80-22

SUMMARY

INSPECTION SUMMARY

♦ INSPECTION ON SEPTEMBER 9-10, (80-17): ROUTINE, UNANNOUNCED INSPECTION OF CORE POWER DISTRIBUTION LIMITS, CORE THERMAL POWER EVALUATION, REACTOR SHUTDOWN MARGIN DETERMINATION, CONTROL ROD WORTH DETERMINATION, CONTROL ROD DRIVE AND POSITION INDICATION CHECKS, REACTIVITY ANOMALIES, AND CYCLE VI STARTUP REPORT. THE INSPECTION INVOLVED 33 INSPECTOR-HOURS ONSITE BY THREE NRC INSPECTORS INCLUDING ZERO INSPECTOR-HOURS ONSITE DURING OFF-SHIFTS. NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

♦ INSPECTION ON AUGUST 1, 4-8, 11-15, 18-22 AND 25-29, (80-18): ROUTINE RESIDENT INSPECTION OF OPERATIONAL SAFETY VERIFICATION; MONTHLY MAINTENANCE OBSERVATION; MONTHLY SURVEILLANCE OBSERVATION; PLANT TRIPS; ASME CODE SECTION FOR PUMPS AND VALVES; AND FOLLOWUP ON REGIONAL REQUESTS. THE INSPECTION INVOLVED 189 INSPECTOR-HOURS ONSITE BY TWO NRC INSPECTORS INCLUDING 28 INSPECTOR-HOURS ONSITE DURING OFF-SHIFTS. OF THE SIX AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED IN FIVE AREAS. ONE ITEM OF NONCOMPLIANCE WAS IDENTIFIED IN ONE AREA (FAILURE TO FULLY IMPLEMENT INSERVICE TESTING OF PUMPS AND VALVES AS REQUIRED).

♦ INSPECTION ON SEPTEMBER 8, (80-19): ROUTINE, UNANNOUNCED INSPECTION OF EMERGENCY PLANNING, INCLUDING OBSERVATION OF DRILL, PRIMARILY FOR COUNTY AND STATE EMERGENCY GOVERNMENT PERSONNEL TO RESPOND TO OFFSITE OPERATIONS, SIMULATE NOTIFICATION OF RESIDENTS, AND TEST COMMUNICATIONS. THE INSPECTION INVOLVED FOUR INSPECTOR-HOURS ONSITE BY ONE NRC INSPECTOR. NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

♦ MEETING ON SEPTEMBER 19, (80-22): A SPECIAL, ANNOUNCED MEETING WAS HELD WITH THE PRESIDENT OF WISCONSIN PUBLIC SERVICE CORPORATION BY THE NRC REGION III DIRECTOR TO DISCUSS WHAT THE NRC PERCEIVED AS A SERIOUS INTERFACE PROBLEM AT THE CORPORATE LEVEL AND TO PURSUE AVENUES TO RESOLVE THE MATTER. THE MEETING LASTED ONE HOUR. THE WISCONSIN PUBLIC SERVICE CORPORATION PRESIDENT INDICATED THAT IT WAS HIS DESIRE TO RESOLVE SUCH A MATTER AND THAT HE WOULD LOOK INTO IT. NO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

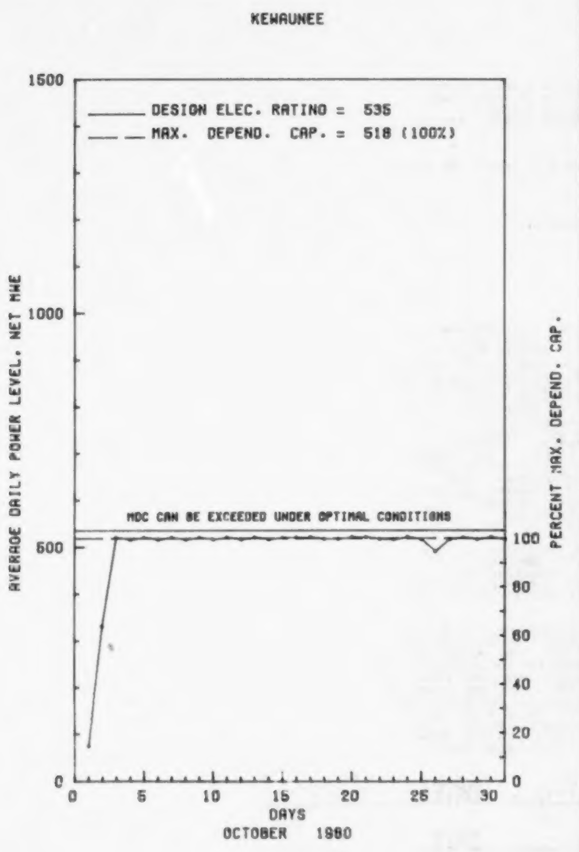
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-13/03L-0	08/22/80	09/19/80	CST LEVEL BELOW TS LIMIT.

REVIEWED BY OMPA LOEB: PCBobe **DATE:** 12/10/80

REVIEWED BY NRR DL: Don Neighbors **DATE:** 12/11/80

KEWAUNEE

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-305
2. Utility Contact: G.RUITER (414) 388-2560 X207
3. Licensed Thermal Power (MWt): 1650
4. Nameplate Rating (Gross MWe): 622 x 0.9 = 560
5. Design Electrical Rating (Net MWe): 535
6. Maximum Dependable Capacity (Gross MWe): 545
7. Maximum Dependable Capacity (Net MWe): 518
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

ITEM 21 CALCULATED BY USING A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	745.0	7,320.0	55,921.0
12. Hours Reactor Was Critical	739.8	5,862.7	46,865.2
13. Reactor Reserve Shutdown Hours	.0	152.7	2,330.5
14. Hours Generator On-Line	732.4	5,761.9	45,757.9
15. Unit Reserve Shutdown Hours	.0	.0	10.0
16. Gross Thermal Energy (MWH)	1,181,104	9,163,685	70,314,808
17. Gross Electrical Energy (MWH)	388,800	3,027,800	23,211,600
18. Net Electrical Energy (MWH)	370,269	2,885,192	22,084,270
19. Unit Service Factor	98.3	78.7	81.8
20. Unit Availability Factor	98.3	78.7	81.8
*21. Unit Capacity Factor (MDC Net)	95.9	75.4	75.4
22. Unit Capacity Factor (DER Net)	92.9	73.7	73.8
23. Unit Forced Outage Rate	1.7	7.4	5.6

24. Shutdowns Scheduled Over Next 6 Months (Type,Date,and Duration of Each):
REFUELING SHUTDOWN - 6 WEEK DURATION - SCHED APRIL 20, 1981.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
9	09/26/80	F	12.6	A	4	80-032	CB		VALVEX RECOVERED FROM FORCED OUTAGE ON 10/01/80.

SUMMARY

KEWAUNEE OPERATED AT OR NEAR FULL POWER FOR THE MONTH OF OCTOBER WITH ONE OUTAGE CONTINUING FROM 9/26 FOR EQUIPMENT FAILURE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C- Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram.
3--Automatic Scram.
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

LACROSSE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Wisconsin/Vernon 19 miles S of LaCrosse
2.	Electric Reliability Council	Mid-Continent Area Reliability Coordination Agreement
3.	Type of Reactor	BWR
4.	Power Level (MWT)	165
5.	Design Electrical Rating (MWE Net)	50
6.	Date of Initial Criticality	July 11, 1967
7.	Date of Commercial Operation	November 1, 1969
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Mississippi River

Utility & Contractor Information

10.	Licensee	Dairyland Power Cooperative
11.	Corporate Address	2615 East Avenue South La Crosse, Wisconsin
12.	Corporate Contact	F. Linder
13.	Architect/Engineer	Sargent & Lundy
14.	Nuclear Steam Supply System	Allis-Chalmers
15.	Constructor	Maxson Construction

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. F. HEISHMAN
18.	IE Section Chief	D. C. BOYD
19.	IE Resident Inspector	W. Forney
20.	Licensing Project Manager	J. J. Shea
21.	Docket Number	50-409
22.	License Number/Date of Issue	DPR-45 issued 8/28/73
23.	Public Document Room	LaCrosse Public Library 800 Main Street LaCrosse, Wisconsin 54601

INSPECTION STATUS

ENFORCEMENT STATUS

A CHANGE TO THE SECURITY PLAN WAS MADE WITHOUT PRIOR COMMISSION APPROVAL.
(8005 DEFICIENCY)

AT TIMES, THE MINIMUM REQUIRED ILLUMINATION LEVEL WAS NOT MAINTAINED.
(8005 INFRACTION)

THE BADGE(S) USED FOR ACCESS CONTROL PURPOSES WAS NOT AS DESCRIBED IN THE SECURITY PLAN.
(8005 DEFICIENCY)

AN EMERGENCY POWER SOURCE FOR SECURITY EQUIPMENT, AS DESCRIBED IN THE SECURITY PLAN, WAS NOT OPERATIONAL.
(8005 INFRACTION)

THE SECONDARY ALARM STATION WAS NOT PROPERLY MANNED.
(8005 INFRACTION)

THE INTRUSION DETECTION SYSTEM DID NOT MEET SECURITY PLAN COMMITMENTS.
(8005 INFRACTION)

PREOPERATIONAL TEST PROCEDURES FOR SOME SECURITY SYSTEMS HAD NOT BEEN COMPLETELY ESTABLISHED.
(8005 INFRACTION)

COMPENSATORY MEASURES EMPLOYED DID NOT MEET SECURITY PLAN COMMITMENTS.
(8005 INFRACTION)

10 CFR SECTION 50.72 REQUIRES, IN PART, LICENSEES OF NUCLEAR POWER REACTORS LICENSED UNDER SECTION 50.21 OR SECTION 50.32 TO NOTIFY THE NRC OPERATIONS CENTER AS SOON AS POSSIBLE AND IN ALL CASES WITHIN ONE HOUR BY TELEPHONE OF ANY EVENT RESULTING IN AUTOMATIC ACTUATION OF THE REACTOR PROTECTION SYSTEM. CONTRARY TO THE ABOVE REQUIREMENT, AT 2:44 P.M. ON AUGUST 22, 1980, THE PLANT EXPERIENCED A REACTOR TRIP, DURING STARTUP AT LOW POWER, DUE TO A LOSS OF FEEDWATER TO THE FORCED CIRCULATING PUMP SCALE. THIS EVENT WAS NOT REPORTED WITHIN THE ONE HOUR TIME LIMIT STATED ABOVE.
(8009 DEFICIENCY)

SECTION 50.59 OF THE CODE OF FEDERAL REGULATIONS REQUIRES THAT, PRIOR TO MAKING CHANGES TO SAFETY RELATED SYSTEMS, THE LICENSE HOLDER MUST PERFORM A SAFETY EVALUATION TO ASSURE THAT THE PROPOSED CHANGE DOES NOT CONSTITUTE AN UNREVIEWED SAFETY QUESTION. TECHNICAL SPECIFICATIONS (TS) 3.5.1.6 AND 3.5.1.7 REQUIRE THE OPERATIONS REVIEW COMMITTEE (ORC) TO REVIEW ALL PROPOSED CHANGES OR MODIFICATIONS TO PLANT SYSTEMS OR EQUIPMENT THAT AFFECTS NUCLEAR SAFETY AND RECOMMEND TO THE PLANT SUPERINTENDENT IN WRITING APPROVAL OR DISAPPROVAL OF THE PROPOSED CHANGE OR MODIFICATION. THIS REVIEW SHALL INCLUDE A DETERMINATION IN WRITING AS TO WHETHER OR NOT THE CHANGE OR MODIFICATION CONSTITUTES AN UNREVIEWED SAFETY QUESTION. CONTRARY TO THE ABOVE, A CHANGE DISCOVERED BY THE RESIDENT INSPECTORS ON JULY 23, 1980, WAS MADE TO THE SHUTDOWN CONDENSER SYSTEM SOMETIME BETWEEN SEPTEMBER 1971 AND JULY 1975 WITHOUT THE PERFORMANCE OF A SAFETY EVALUATION OR A REVIEW BY THE ORC. ADDITIONALLY, ANOTHER FACILITY CHANGE WAS MADE ON JULY 23, 1980 WITHOUT THE PERFORMANCE OF A SAFETY EVALUATION OR REVIEW BY THE ORC WHEN THE ABOVE UNAPPROVED MODIFICATION WAS REMOVED.
(8009 INFRACTION)

TECHNICAL SPECIFICATION 4.2.1.1 REQUIRES THAT CONTAINMENT INTEGRITY TO BE MAINTAINED WHENEVER THE REACTOR IS OPERATING AT POWER. CONTRARY TO THE ABOVE, ON JULY 23, 1980, WITH THE REACTOR AT 85% POWER, CONTAINMENT INTEGRITY WAS VIOLATED FOR APPROXIMATELY ONE MINUTE WHILE AN UNAUTHORIZED MODIFICATION WAS REMOVED FROM THE SHELL SIDE OF THE SHUTDOWN CONDENSER AND REPLACED WITH A PIPE CAP.
(8009 INFRACTION)

TECHNICAL SPECIFICATION 2.1.2.5(1) REQUIRES THAT ANY PIPING SYSTEM THAT IS OPEN TO CONTAINMENT BUILDING ATMOSPHERE AND OUTSIDE ATMOSPHERE SHALL BE PROVIDED WITH TWO VALVES IN SERIES. CONTRARY TO THE ABOVE, AN UNAUTHORIZED MODIFICATION WAS MADE TO THE SHUTDOWN CONDENSER SHELL SIDE VENT, SOMETIME BETWEEN SEPTEMBER 1971 AND JULY 1975, WHICH RESULTED IN A PIPING SYSTEM THAT DID NOT MEET THE REQUIRED TWO VALVE CRITERIA, IN THAT ONLY ONE NORMALLY SHUT VALVE WAS PROVIDED.
(8009 INFRACTION)

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE.			

==

INSPECTION STATUS

OTHER ITEMS OF INTEREST

FACILITY ITEMS

- DURING THE MONTH OF AUGUST THE FACILITY HAS EXPERIENCED SEVERAL LEAKS OF THE MECHANICAL SEALS ON CONTROL ROD DRIVES. THE REPAIR OF THESE LEAKS RESULTED IN FORCED PLANT SHUTDOWNS. THE SEAL ON NO. 1A FORCED RECIRCULATION PUMP WAS AGAIN REPLACED DURING THE AUGUST OUTAGE. (80-09)

MANAGERIAL ITEMS

- LACBWR HAS HIRED A QUALIFIED MECHANICAL ENGINEER WHO WILL REPORT FOR DUTY ON OCTOBER 1, 1980. (80-09)

PLANT STATUS

- OPERATING IN COASTDOWN MODE. REFUELING OUTAGE TO BEGIN ON NOVEMBER 14, 1980.

- LAST IE SITE INSPECTION DATE: AUGUST 1-31, 1980

- INSPECTION REPORT NO: 80-09

INSPECTION SUMMARY

- INSPECTION ON JULY 7-11, 28 AND AUGUST 28-29, (80-05): INCLUDED A REVIEW OF THE SECURITY PLAN; SECURITY PROGRAM AUDIT; SECURITY ORGANIZATION; PROTECTED AREA PHYSICAL BARRIERS; VITAL AREA PHYSICAL BARRIERS; LIGHTING; ACCESS CONTROL (IDENTIFICATION, AUTHORIZATION, AND BADGING); ACCESS CONTROL (SEARCH); ACCESS CONTROL (ESCORTING); ACCESS CONTROL (VITAL AREAS); ALARM STATIONS; DETECTION AIDS; ASSESSMENT AIDS; COMMUNICATIONS; TESTING AND MAINTENANCE; COMPENSATORY MEASURES; POWER SUPPLY; RESPONSE; LOCKS, KEYS, AND COMBINATIONS; RECORDS AND REPORTS; LICENSEE ACTION OF PREVIOUS INSPECTION FINDINGS; FOLLOWUP ON UNRESOLVED ITEMS AND INDEPENDENT INSPECTION EFFORT. THE INSPECTIONS INVOLVED 99 INSPECTOR-HOURS ONSITE BY TWO NRC INSPECTORS. THE INSPECTIONS BEGAN DURING THE DAY SHIFT; 21 INSPECTOR-HOURS WERE ACCOMPLISHED DURING OFF-SHIFT PERIODS. THE LICENSEE WAS FOUND TO BE IN COMPLIANCE WITH NRC REQUIREMENTS WITHIN THE 23 AREAS EXAMINED DURING THE INSPECTION EXCEPT FOR THE FOLLOWING EIGHT ITEMS: SECURITY PLAN - A CHANGE TO THE SECURITY PLAN, WHICH DECREASED THE SAFEGUARDS EFFECTIVENESS, WAS MADE WITHOUT PRIOR COMMISSION APPROVAL; LIGHTING - THE REQUIRED ILLUMINATION LEVEL WAS NOT MAINTAINED AT ALL TIMES; ACCESS CONTROL (VITAL AREAS) - ACCESS TO VITAL AREAS WAS NOT CONTROLLED AS REQUIRED BY THE SECURITY PLAN; ALARM STATIONS - THE SECONDARY ALARM STATION WAS NOT CONTINUOUSLY MANNED; DETECTION AIDS - A PERIMETER INTRUSION DETECTION SYSTEM WAS NOT OPERABLE; COMPENSATORY MEASURES - COMPENSATORY MEASURES DID NOT PROVIDE AN EQUIVALENT LEVEL OF PROTECTION IN THE EVENT OF AN OUTAGE OF SOME SECURITY HARDWARE; AND POWER SUPPLY - AN ONSITE SOURCE OF EMERGENCY POWER FOR ALL SECURITY SYSTEMS WAS NOT PROVIDED.

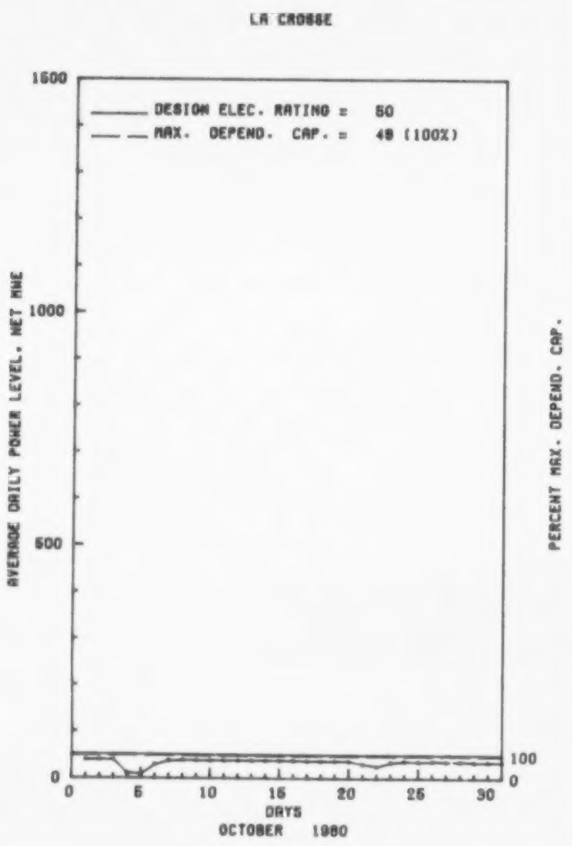
- INSPECTION ON AUGUST 1-31, (80-09): ROUTINE, ANNOUNCED INSPECTION OF THE LICENSEE'S OPERATIONAL SAFETY; SURVEILLANCE; MAINTENANCE; FOLLOWUP ACTION TO IE BULLETINS, IE CIRCULARS AND OPEN INSPECTION ITEMS. THE INSPECTION INVOLVED 300 INSPECTOR-HOURS ONSITE BY FOUR NRC INSPECTORS INCLUDING 32 INSPECTOR HOURS ONSITE DURING OFF-SHIFTS. OF THE SEVEN AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE FOUND IN SIX AREAS; FOUR ITEMS OF NONCOMPLIANCE WERE FOUND IN ONE AREA (FAILURE TO ACCOMPLISH ORC REVIEW AND APPROVAL OF A FACILITY CHANGE; FAILURE TO PROVIDE TWO MANUAL VALVES ON A SYSTEM EXPOSED TO BOTH CONTAINMENT AND OUTSIDE ATMOSPHERE; VIOLATING CONTAINMENT INTEGRITY WHILE REACTOR WAS AT POWER; AND FAILURE TO MAKE PROMPT TELEPHONE NOTIFICATION TO NRC OPERATIONS CENTER OF A SIGNIFICANT EVENT).

REVIEWED BY OMPA LOEB: John J. Briggs **DATE:** 12-10-80

REVIEWED BY NRR DL: **DATE:** 12-11-80

LACROSSE

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-409
2. Utility Contact: L. S. Goodman (608) 689-2331
3. Licensed Thermal Power (MWt): 165
4. Nameplate Rating (Gross MWe): 76.8 x 0.85 =65
5. Design Electrical Rating (Net MWe): 50
6. Maximum Dependable Capacity (Gross MWe): 50
7. Maximum Dependable Capacity (Net MWe): 48
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	745.0	7,320.0	96,435.0
12. Hours Reactor Was Critical	730.0	6,077.6	63,883.7
13. Reactor Reserve Shutdown Hours	.0	.0	478.0
14. Hours Generator On-Line	717.7	5,813.1	58,818.1
15. Unit Reserve Shutdown Hours	.0	.0	79.0
16. Gross Thermal Energy (MWH)	94,969	772,917	8,057,696
17. Gross Electrical Energy (MWH)	27,635	223,820	2,422,952
18. Net Electrical Energy (MWH)	25,770	208,205	2,240,948
19. Unit Service Factor	96.3	79.4	61.0
20. Unit Availability Factor	96.3	79.4	61.1
21. Unit Capacity Factor (MDC Net)	72.1	59.3	48.4
22. Unit Capacity Factor (DER Net)	69.2	56.9	46.5
23. Unit Forced Outage Rate	3.7	13.4	7.1

24. Shutdowns Scheduled Over Next 6 Months (Type,Date,and Duration of Each): REFUEL OUTAGE, 11/09/80; EST OPER LICENSING EXAMS, 01/12/81.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-10	10/04/80	F	27.3	A	3		CH	INSTRU	FAILURE OF 1B REACTOR FEEDPUMP CONTROLLER AMPLIFIER CAUSED PUMP TO FAIL HIGH, CAUSING REACTOR SCRAM ON HIGH WATER LEVEL. SPARE AMPLIFIER WAS INSTALLED AND TESTED WITH SATISFACTORY RESULTS. CAUSE BELIEVED DUE TO WATER-DETERGENT MIXTURE WHICH SPLASHED ONTO CONTROLLERS AUGUST 22, 1980.
80-11	10/21/80	F	.0	B	5		HC	XXXXXX	A POWER REDUCTION FROM 81% TO 54% WAS CONDUCTED FOR THE PURPOSE OF REMOVING EACH OF THE TWO TURBINE CONDENSER WATER BOXES TO CLEAN CONDENSER TUBES AND TO REMOVE ACCUMULATED SILT FROM THE WATER BOXES. THE CONDENSER CLEANING WAS ACCOMPLISHED TO IMPROVE TURBINE EXHAUST VACUUM AND TEMPERATURE CONDITIONS.

SUMMARY

LA CROSSE OPERATED IN END-OF-CYCLE COASTDOWN DURING OCTOBER IN PREPARATION FOR NOVEMBER REFUELING, WITH ONE OUTAGE FOR EQUIPMENT FAILURE ON THE 4TH, AND ONE REDUCTION ON THE 27TH FOR MAINTENANCE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

MAINE YANKEE

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Maine/Lincoln 10 miles N of Bath (near Wiscasset)
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2630
5.	Design Electrical Rating (MWE Net)	825
6.	Date of Initial Criticality	October 23, 1972
7.	Date of Commercial Operation	December 28, 1972
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Back River (Atlantic)

Utility & Contractor Information

10.	Licensee	Maine Yankee Atomic Power Co.
11.	Corporate Address	20 Turnpike Road Westboro, Massachusetts 01581
12.	Corporate Contact	D. W. Edwards, Assistant to Vice President
13.	Architect/Engineer	Stone & Webster
14.	Nuclear Steam Supply System	Combustion Engineering
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	IE Section Chief	T. T. Martin
19.	IE Resident Inspector	W. J. LAZARUS (207) 882-7519
20.	Licensing Project Manager	G. Requa (X28478)
21.	Docket Number	50-309
22.	License Number/Date of Issue	DPR-36, 9/15/72
23.	Public Document Room	Wiscasset Public Library High Street Wiscasset, Maine 04578

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO 10 CFR 50, APPENDIX B, CRITERION V AND XII, AND ANSI N18.7-1976, PARAGRAPH 5.3.7, CARDOX SYSTEM PRESSURE AND LEVEL GAGES AND DIESEL FIRE PUMP FUEL OIL TANK LEVEL GAGES WERE NOT BEING CALIBRATED; AND CONVERSION CHART FROM LEVEL TO GALLONS FOR DIESEL GENERATOR FUEL OIL TANK LEVELS AND PRIMARY WATER STORAGE TANK LEVEL ACCURACY AND CORRECTNESS COULD NOT BE VERIFIED NOR CONVERSION CHARTS ESTABLISHED AS PART OF AN APPROVED PROCEDURE.
(8012 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEM AND COMPONENT PROBLEMS:

- FAILURE OF A CONTROL ELEMENT DRIVE MECHANISM (CEDM) RESULTED IN AN IMMOVABLE ROD. THE CEDM WAS REPAIRED AND THE ROD WAS RETURNED TO SERVICE.

PLANT STATUS:

- THE PLANT OPERATED AT FULL POWER EXCEPT FOR ONE SHORT LOAD REDUCTION FOR CORRECTIVE ACTIONS ON AN IMMOVABLE CONTROL ROD.
- LAST IE SITE INSPECTION DATE: 10/3-31/80
- INSPECTION REPORT NO: 50-309/80-16

SUMMARY

INSPECTION SUMMARY

- 50-309/80-11 - AUG 18 - SEP 1: ROUTINE, ONSITE, REGULAR AND BACKSHIFT INSPECTION BY TWO RESIDENT INSPECTORS (55 HRS). AREAS INSPECTED INCLUDED THE CONTROL ROOM, TURBINE BUILDING, PRIMARY AUXILIARY BUILDING, SPENT FUEL BUILDING, SPRAY BUILDING AND AUXILIARY FEED PUMP AREAS. ACTIVITIES/RECORDS INSPECTED INCLUDED RADIATION PROTECTION, PHYSICAL SECURITY, PLANT OPERATIONS, MAINTENANCE, AND SURVEILLANCE TESTING. NO NONCOMPLIANCES WERE IDENTIFIED.
- 50-309/80-12 - AUG 25-29: ROUTINE, UNANNOUNCED INSPECTION BY ONE REGION-BASED INSPECTOR (32 HRS) OF LICENSEE ACTION ON PREVIOUS INSPECTION FINDINGS; ADMINISTRATIVE CONTROL OF SAFETY RELATED CALIBRATIONS; SURVEILLANCE CALIBRATIONS OF SAFETY RELATED COMPONENTS AND EQUIPMENT REQUIRED BY TECHNICAL SPECIFICATIONS; CALIBRATIONS REQUIRED BY TECHNICAL SPECIFICATIONS OF COMPONENTS AND EQUIPMENT ASSOCIATED WITH SAFETY RELATED SYSTEMS AND/OR FUNCTIONS; CALIBRATION AND CONTROL OF TEST EQUIPMENT; WITNESSING OF CALIBRATIONS; TECHNICIAN QUALIFICATIONS; AND A FACILITY TOUR. TWO NONCOMPLIANCES WERE IDENTIFIED: FAILURE TO PROVIDE PROCEDURES FOR AND PERFORM CALIBRATIONS OF SAFETY-RELATED INSTRUMENTS; FAILURE TO REVIEW AND APPROVE DOCUMENTS USED FOR THE VERIFICATION OF SAFETY-RELATED PARAMETERS.

REPORTS RECEIVED FROM LICENSEE

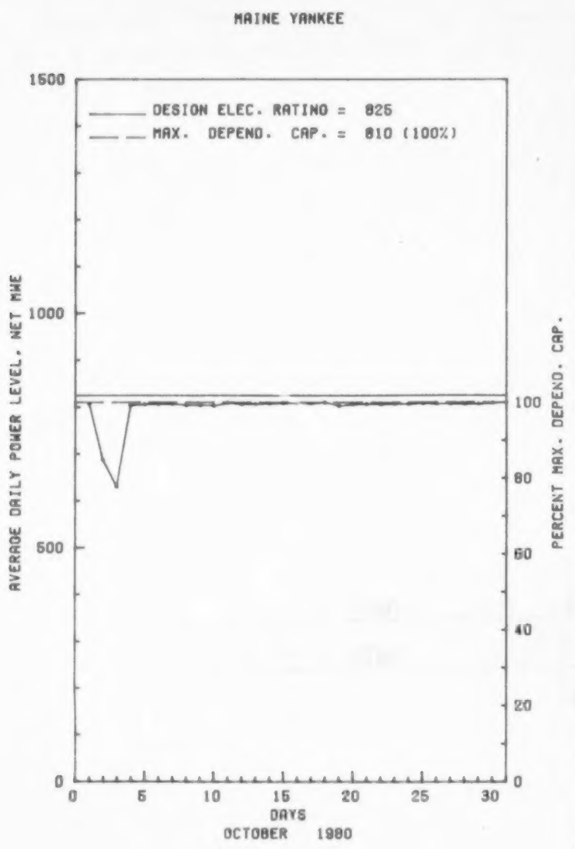
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-017/ 01T	08/22/80	10/02/80	RWST BORON CONCENTRATION BELOW SPECIFICATION
80-018/ 03L	08/26/80	09/22/80	FAILURE OF DIESEL GENERATOR DG1B

REVIEWED BY OMPA LOEB: MR Beebe **DATE:** 12/10/80

REVIEWED BY NRR DL: CC Nelson **DATE:** 12/11/80

MAINE YANKEE

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-309
2. Utility Contact: F.C. BEERS (617) 366-9011
3. Licensed Thermal Power (MWt): 2630
4. Nameplate Rating (Gross MWe): 900 x 0.9 = 810
5. Design Electrical Rating (Net MWe): 825
6. Maximum Dependable Capacity (Gross MWe): 850
7. Maximum Dependable Capacity (Net MWe): 810
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

CAPACITY FACTORS CALCULATED WITH A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	745.0	7,320.0	69,948.6
12. Hours Reactor Was Critical	745.0	5,413.3	56,623.2
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	745.0	5,214.1	54,731.8
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,886,229	11,400,303	116,612,386
17. Gross Electrical Energy (MWH)	622,020	3,728,530	38,301,710
18. Net Electrical Energy (MWH)	593,934	3,545,744	36,372,930
19. Unit Service Factor	100.0	71.2	78.2
20. Unit Availability Factor	100.0	71.2	78.2
*21. Unit Capacity Factor (MDC Net)	98.4	59.8	67.3
22. Unit Capacity Factor (DER Net)	96.6	58.7	65.1
23. Unit Forced Outage Rate	.0	9.0	7.1

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	10/02/80	F	1.5	A	5		RB	CRDRVE	DURING CEA EXERCISES, CEA #8 WOULD NOT MOVE AND A LOAD REDUCTION COMMENCED. WHILE TROUBLESHOOTING CEA #8, IT DROPPED INTO THE CORE. POWER WAS STABILIZED AT 78%. PROBLEM IN CEA #8 CONTROL CIRCUITRY WAS FOUND AND CORRECTED.
	10/02/80	F	23.4	B	5		CH	PUMPXX	WHILE LOAD WAS REDUCED DUE TO CEA #8 PROBLEMS, IT WAS DECIDED TO DROP LOAD FURTHER TO ADD PACKING TO HEATER DRAIN PUMP P-63A AND REPLACE THE THRUST COLLAR ON FEED-WATER PUMP P-2A. LOAD WAS DROPPED TO APPROXIMATELY 50% WHILE P-62A WAS OOS AND THEN INCREASED TO APPROXIMATELY 60% WHILE COMPLETING REPAIRS TO P-2A.

SUMMARY

MAINE YANKEE OPERATED AT OR NEAR FULL POWER FOR OCTOBER, WITH TWO REDUCTIONS FOR EQUIPMENT FAILURE AND MAINTENANCE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram.
3--Automatic Scram.
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

MILLSTONE 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Connecticut/New London 5 miles SW of New London
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2011
5.	Design Electrical Rating (MWE Net)	660
6.	Date of Initial Criticality	October 26, 1970
7.	Date of Commercial Operation	March 1971
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Long Island Sound

Utility & Contractor Information

10.	Licensee	Northeast Nuclear Energy Company
11.	Corporate Address	P. O. Box 270 Hartford, Connecticut
12.	Corporate Contact	D. Switzer, President
13.	Architect/Engineer	Ebasco
14.	Nuclear Steam Supply System	GE
15.	Constructor	Ebasco

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	IE Section Chief	R. KEIMIG
19.	IE Resident Inspector	J. SHEDLOSKY (203-447-3179) FTS: 842-7287
20.	Licensing Project Manager	J. J. Shea
21.	Docket Number	50-245
22.	License Number/Date of Issue	DPR-21, 10/7/70
23.	Public Document Room	Waterford Public Library Rope Ferry Road Route 156 Waterford, Connecticut 06385

M-4

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS:

- THE PLANT WAS SHUTDOWN OCTOBER 4 FOR A TEN WEEK REFUEL OUTAGE.
- LAST IE SITE INSPECTION DATE: 10/27-31/80
- INSPECTION REPORT NO: 50-245/80-21

SUMMARY

INSPECTION SUMMARY

- 50-245/80-13 - AUG 3 - SEP 6: ROUTINE, ONSITE, REGULAR AND BACKSHIFT INSPECTION BY TWO RESIDENT INSPECTORS (43 HRS). AREAS INCLUDED THE CONTROL ROOM AND THE ACCESSIBLE PORTIONS OF THE REACTOR, TURBINE, RADIOACTIVE WASTE, GAS TURBINE GENERATOR, AND INTAKE BUILDINGS; RADIATION PROTECTION; PHYSICAL SECURITY; FIRE PROTECTION; PLANT OPERATING RECORDS; SURVEILLANCE TESTING; CALIBRATION; MAINTENANCE; CORE POWER DISTRIBUTION LIMITS; AND REPORTING TO THE NRC. TWO NONCOMPLIANCES WERE IDENTIFIED: FAILURE TO MAKE AIRBORNE RADIOACTIVE MATERIAL SURVEYS; FAILURE TO PREPARE PROCEDURES CONSISTENT WITH 10 CFR 20.103.
- 50-245/80-15 - SEP 11: SPECIAL, ANNOUNCED MANAGEMENT MEETING TO DISCUSS THE RESULTS OF AN APPRAISAL BOARD CONVENED TO EVALUATE THE PERFORMANCE OF THE LICENSEE FROM JULY 1979 TO JULY 1980. AREAS ADDRESSED INCLUDED: PLANT OPERATIONS, REFUELING OPERATIONS, RADIATION PROTECTION, RADWASTE OPERATIONS, TRANSPORTATION, MAINTENANCE, SECURITY AND SAFEGUARDS, SURVEILLANCE AND PRE-OP TESTING, DESIGN CHANGE AND MODIFICATIONS, EMERGENCY PLANNING, ENVIRONMENTAL, TRAINING, MANAGEMENT CONTROLS, FIRE PROTECTION, QA/QC, COMMITTEE ACTIVITIES/AUDITS, REPORTING, PROCUREMENT. A SUMMARY OF THE LICENSEE PERFORMANCE EVALUATION WAS PRESENTED. AREAS OF CONCERN WERE DISCUSSED WITH CORPORATE MANAGEMENT. NO NEW ENFORCEMENT ACTIONS WERE IDENTIFIED. A NUMBER OF AREAS WHERE IMPROVEMENTS APPEAR ACHIEVABLE WERE IDENTIFIED.
- 50-245/80-17 - SEP 7 - OCT 4: ROUTINE, ONSITE, REGULAR AND BACKSHIFT INSPECTION BY TWO RESIDENT INSPECTORS (55.5 HRS). AREAS INSPECTED INCLUDED THE CONTROL ROOM AND THE ACCESSIBLE PORTIONS OF THE REACTOR, TURBINE, RADIOACTIVE WASTE, GAS TURBINE GENERATOR, AND INTAKE BUILDINGS; RADIATION PROTECTION; PHYSICAL SECURITY; FIRE PROTECTION; PLANT OPERATING RECORDS; SURVEILLANCE TESTING; CALIBRATION; MAINTENANCE; CORE POWER DISTRIBUTION LIMITS; AND REPORTING TO THE NRC. NO NONCOMPLIANCES WERE IDENTIFIED.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

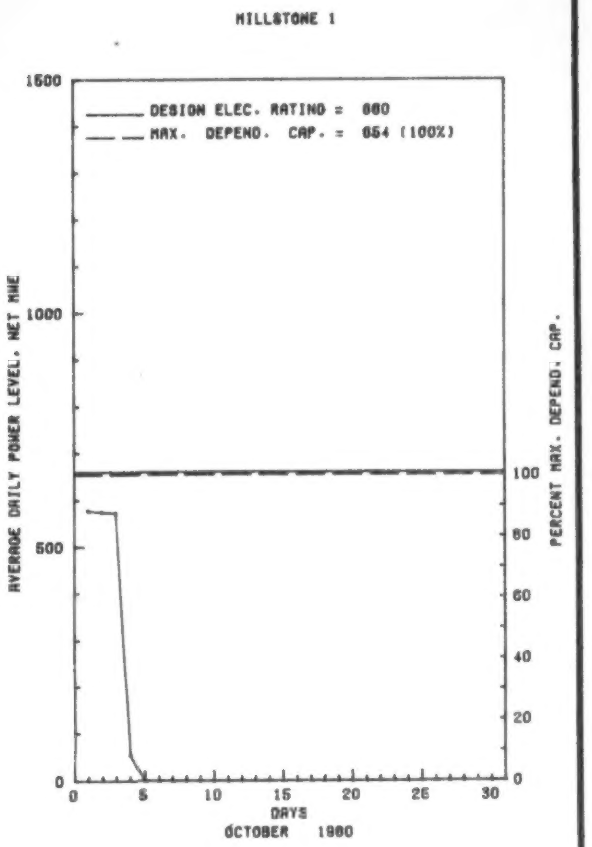
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-001E	08/12/80	10/03/80	DETECTION OF ISOTOPES OF COBALT AND SILVER IN OYSTERS WITHIN 500 FT OF THE DISCHARGE AT LEVELS GREATER THAN A FACTOR OF TEN OF CONTROL STATION LEVELS.
80-013/03L	09/08/80	10/03/80	SETPOINT DRIFT ONE OF FOUR DRYWELL HIGH PRESSURE RPS TRIP SWITCHES.
80-014/01T	10/05/80	10/16/80	TWO MSIV EXCEEDED TECHNICAL SPECIFICATION LIMITS FOR VALVE LEAKAGE.

REVIEWED BY OMPA LOEB: John J. Rizzo DATE: 12-10-80

REVIEWED BY NRR DL: J J Shea DATE: 12.11.80

MILLSTONE 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-245
2. Utility Contact: G. HARRAN (203) 447-1792 X455
3. Licensed Thermal Power (MWt): 2011
4. Nameplate Rating (Gross MWe): 735 x 0.9 = 662
5. Design Electrical Rating (Net MWe): 660
6. Maximum Dependable Capacity (Gross MWe): 684
7. Maximum Dependable Capacity (Net MWe): 654
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	745.0	7,320.0	87,000.0
12. Hours Reactor Was Critical	88.3	6,074.3	66,459.3
13. Reactor Reserve Shutdown Hours	.0	592.3	1,527.3
14. Hours Generator On-Line	76.3	6,030.3	64,126.2
15. Unit Reserve Shutdown Hours	.0	26.5	26.5
16. Gross Thermal Energy (MWH)	22,392	10,382,182	114,430,127
17. Gross Electrical Energy (MWH)	45,000	3,564,500	38,795,896
18. Net Electrical Energy (MWH)	39,950	3,394,669	37,034,431
19. Unit Service Factor	10.2	82.4	73.7
20. Unit Availability Factor	10.2	82.7	73.7
21. Unit Capacity Factor (MDC Net)	8.2	70.9	65.1
22. Unit Capacity Factor (DER Net)	8.1	70.3	64.5
23. Unit Forced Outage Rate	.0	.2	15.3

24. Shutdowns Scheduled Over Next 6 Months (Type,Date,and Duration of Each): NONE

25. If Shut Down At End of Report Period, Estimated Date of Startup: 12/19/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
10	10/04/80	S	668.7	C	1				UNIT WAS REMOVED FOR ANNUAL REFUEL AND MAINTENANCE OUTAGE.

SUMMARY

MILLSTONE 1 OPERATED ON-LINE THRU THE 4TH OF OCTOBER, WHEN IT COMMENCED UPON A REFUELING AND MAINTENANCE OUTAGE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

MILLSTONE 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1. Location (state/county) (nearest population center)	Connecticut/New London 5 miles SW of New London
2. Electric Reliability Council	Northeast Power Coordinating Council
3. Type of Reactor	PWR
4. Power Level (MWT)	2700
5. Design Electrical Rating (MWE Net)	870
6. Date of Initial Criticality	October 17, 1975
7. Date of Commercial Operation	December 26, 1975
8. Condenser Cooling Method	Once Through
9. Condenser Cooling Water Source	Long Island Sound

Utility & Contractor Information

10. Licensee	Northeast Nuclear Energy Company
11. Corporate Address	P. O. Box 270 Hartford, Connecticut
12. Corporate Contact	D. Switzer, President
13. Architect/Engineer	Bechtel
14. Nuclear Steam Supply System	Combustion Engineering
15. Constructor	Bechtel

Regulatory Information

16. IE Region responsible	I
17. IE Branch Chief	E. Brunner
18. IE Section Chief	R. Keimig
19. IE Resident Inspector	J. T. SHEDLOSKY (203-447-3179) FTS: 642-7287
20. Licensing Project Manager	R. Zimmerman E. L. Conner, Jr.
21. Docket Number	50-336
22. License Number/Date of Issue	DPR-65, 8/1/75
23. Public Document Room	Waterford Public Library Rope Ferry Road Route 156 Waterford, Connecticut 06385

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS:

- REACTOR CRITICALITY WAS ACHIEVED ON OCTOBER 13 FOLLOWING THE COMPLETION OF AN EIGHT WEEK REFUELING OUTAGE. THE UNIT WAS AT 100% POWER ON OCTOBER 28.

- LAST IE SITE INSPECTION DATE: 10/5-31/80

- INSPECTION REPORT NO: 50-336/80-20

SUMMARY

INSPECTION SUMMARY

- 50-336/80-12 - AUG 26-28: ROUTINE, UNANNOUNCED INSPECTION BY ONE REGION-BASED INSPECTOR (31 HRS) OF RADIATION PROTECTION PROGRAM ASSOCIATED WITH THE REFUELING OUTAGE, INCLUDING: RADIATION PROTECTION PROCEDURES; ADVANCED PLANNING AND PREPARATION; EXPOSURE CONTROL; POSTING AND CONTROL; INSTRUMENTS AND EQUIPMENT; SURVEYS; AND PREVIOUS ITEMS OF NONCOMPLIANCE. IN ADDITION, THE LICENSEE'S RESPONSE TO IE BULLETIN NOS. 78-07, 78-08, AND 80-10 WERE REVIEWED. NO NONCOMPLIANCES WERE IDENTIFIED.

- 50-336/80-14 - AUG 3 - SEP 6: ROUTINE, ONSITE, REGULAR AND BACKSHIFT INSPECTION BY TWO RESIDENT INSPECTORS (55 HRS). AREAS INSPECTED INCLUDED THE CONTROL ROOM, PRIMARY CONTAINMENT ENCLOSURE, AUXILIARY, TURBINE AND INTAKE BUILDINGS; AND THE CONDENSATE POLISHING FACILITY; RADIATION PROTECTION; PHYSICAL SECURITY; FIRE PROTECTION; PLANT OPERATING RECORDS; SURVEILLANCE TESTING; CALIBRATION; MAINTENANCE; CORE POWER DISTRIBUTION LIMITS; AND REPORTING TO THE NRC. NO NONCOMPLIANCES WERE IDENTIFIED.

- 50-336/80-16 - SEP 11: SPECIAL, ANNOUNCED MANAGEMENT MEETING TO DISCUSS THE RESULTS OF AN APPRAISAL BOARD CONVENED TO EVALUATE THE PERFORMANCE OF THE LICENSEE FROM JULY 1979 TO JULY 1980. AREAS ADDRESSED INCLUDED: PLANT OPERATIONS, REFUELING OPERATIONS, RADIATION PROTECTION, RADWASTE OPERATIONS, TRANSPORTATION, MAINTENANCE, SECURITY AND SAFEGUARDS, SURVEILLANCE AND PRE-OP TESTING, DESIGN CHANGE AND MODIFICATIONS, EMERGENCY PLANNING, ENVIRONMENTAL, TRAINING, MANAGEMENT CONTROLS, FIRE PROTECTION, QA/QC, COMMITTEE ACTIVITIES/AUDITS, REPORTING, PROCUREMENT. A SUMMARY OF THE LICENSEE PERFORMANCE EVALUATION WAS PRESENTED. AREAS OF CONCERN WERE DISCUSSED WITH CORPORATE MANAGEMENT. NO NEW ENFORCEMENT ACTIONS WERE IDENTIFIED. A NUMBER OF AREAS WHERE IMPROVEMENTS APPEAR ACHIEVABLE WERE IDENTIFIED.

- 50-336/80-18 - SEP 23-26: ROUTINE, ANNOUNCED INSPECTION BY A REGION-BASED INSPECTOR (26 HRS) OF THE FUEL HANDLING FAN ASSOCIATED WITH THE AUXILIARY BUILDING VENTILATION SYSTEM; REACTOR COOLANT PUMPS ASSOCIATED WITH THE RBCCW AND THE BORIC ACID TANK HEATERS ASSOCIATED WITH THE CHEMICAL VOLUME AND CONTROL SYSTEM TO VERIFY COMPONENT INSTALLATION AS IDENTIFIED AND REPORTED BY THE LICENSEE IN RESPONSE TO NRC BULLETIN NO. 79-01B. NO NONCOMPLIANCES WERE IDENTIFIED.

- 50-336/80-19 - SEP 7 - OCT 4: ROUTINE, ONSITE, REGULAR AND BACKSHIFT INSPECTION BY TWO RESIDENT INSPECTORS (86 HRS). AREAS INSPECTED INCLUDED THE CONTROL ROOM, PRIMARY CONTAINMENT ENCLOSURE, AUXILIARY, TURBINE AND INTAKE BUILDINGS; AND THE CONDENSATE POLISHING FACILITY; RADIATION PROTECTION; PHYSICAL SECURITY; FIRE PROTECTION; PLANT OPERATING RECORDS; SURVEILLANCE TESTING; CALIBRATION; MAINTENANCE; CORE POWER DISTRIBUTION LIMITS; AND REPORTING TO THE NRC. NO NONCOMPLIANCES WERE IDENTIFIED.

- 50-336/80-21 - SEP 30 - OCT 3: ROUTINE, UNANNOUNCED INSPECTION BY A REGION-BASED INSPECTOR (20 HRS) OF INSERVICE INSPECTION ACTIVITIES. NO NONCOMPLIANCES WERE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

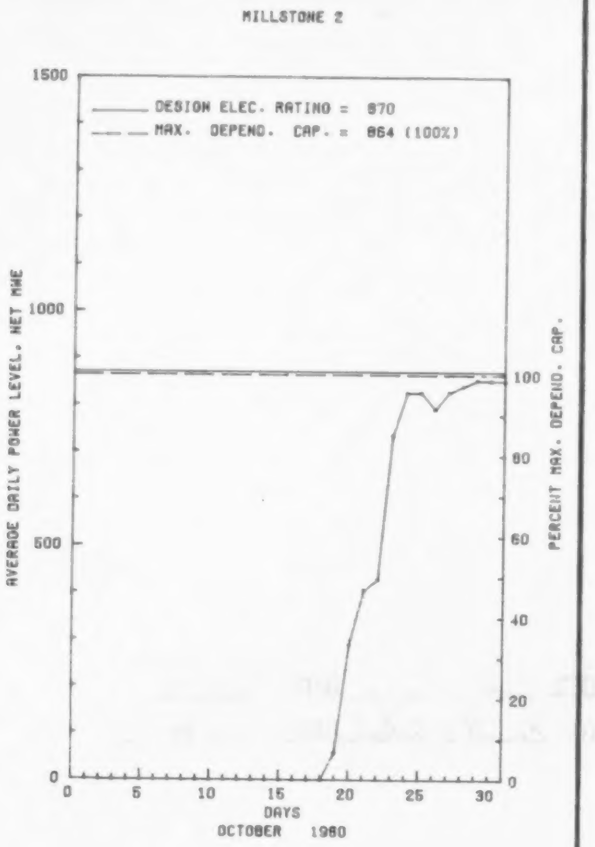
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-001E	08/12/80	10/03/80	DETECTION OF ISOTOPES OF COBALT AND SILVER IN OYSTERS WITHIN 500 FT OF THE DISCHARGE AT LEVELS GREATER THAN A FACTOR OF TEN OF CONTROL STATION LEVELS.
80-031/03L	08/17/80	09/12/80	WITH TWO OF FOUR WIDE RANGE NEUTRON FLUX INSTRUMENTS TAGGED OUT FOR MAINTENANCE THE PREAMPLIFIER ON ONE OF THE TWO OPERABLE CHANNELS FAILED. THE REACTOR WAS IN COLD SHUTDOWN.

REVIEWED BY OMPA LOEB: AmR Beebe **DATE:** 12/10/80

REVIEWED BY NRR DL: E L Conner **DATE:** 12/11/80

MILLSTONE 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-336
2. Utility Contact: G. H. Howlett (203) 447-1791 x364
3. Licensed Thermal Power (MWt): 2700
4. Nameplate Rating (Gross MWe): 1011 x 0.9 = 910
5. Design Electrical Rating (Net MWe): 870
6. Maximum Dependable Capacity (Gross MWe): 895
7. Maximum Dependable Capacity (Net MWe): 864
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

CAPACITY FACTORS CALCULATED WITH A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	745.0	7,320.0	42,528.0
12. Hours Reactor Was Critical	429.5	4,706.2	29,784.9
13. Reactor Reserve Shutdown Hours	4.5	4.5	2,076.9
14. Hours Generator On-Line	301.2	4,484.8	28,305.0
15. Unit Reserve Shutdown Hours	132.8	132.8	468.2
16. Gross Thermal Energy (MWH)	659,001	11,679,277	69,377,985
17. Gross Electrical Energy (MWH)	215,600	3,829,672	22,414,317
18. Net Electrical Energy (MWH)	200,139	3,668,534	21,448,792
19. Unit Service Factor	40.4	61.3	66.6
20. Unit Availability Factor	58.3	63.1	67.7
21. Unit Capacity Factor (MDC Net)	31.1	58.0	61.1
22. Unit Capacity Factor (DER Net)	30.9	57.6	60.0
23. Unit Forced Outage Rate	.0	22.4	23.6

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWN/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
13	08/16/80	S	442.5	C	4				CYCLE 4 REFUEL OUTAGE CONTINUES.
14	10/19/80	S	1.3	B	1				POWER WAS MANUALLY REDUCED TO APPROXIMATELY 8% AT WHICH TIME A TURBINE OVERSPEED TRIP TEST WAS CONDUCTED WITH THE REACTOR REMAINING AT POWER. GENERATOR OFF-LINE.

SUMMARY

CYCLE 4 REFUEL OUTAGE CONCLUDED WITH THE REACTOR BEING BROUGHT CRITICAL ON THE 13TH. LOW POWER PHYSICS TESTING WAS CONDUCTED THROUGH THE 18TH WITH THE UNIT BEING BROUGHT ON-LINE ON THE 19TH. 100% POWER OPERATIONS WAS ACHIEVED ON THE 29TH. POWER ASCENSION TESTING IS STILL ONGOING AT THE END OF THIS REPORTING PERIOD.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

MONTICELLO

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Minnesota/Wright 30 miles NW of Minneapolis
2.	Electric Reliability Council	Mid-Continent Area Reliability Coordination Agreement
3.	Type of Reactor	BWR
4.	Power Level (MWT)	1670
5.	Design Electrical Rating (MWE Net)	545
6.	Date of Initial Criticality	December 10, 1970
7.	Date of Commercial Operation	June 30, 1971
8.	Condenser Cooling Method	Once Through with Cooling Tower
9.	Condenser Cooling Water Source	Mississippi River

Utility & Contractor Information

10.	Licensee	Northern States Power Co.
11.	Corporate Address	414 Nicollet Mall - 8th Floor Minneapolis, Minnesota 55401
12.	Corporate Contact	G. H. Neils (Oper.) L. O. Meyer (Lic.)
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	GE
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. F. HEISHMAN
18.	IE Section Chief	W. S. Little
19.	IE Resident Inspector	C. Brown
20.	Licensing Project Manager	K. Eccleston
21.	Docket Number	50-263
22.	License Number/Date of Issue	DPR-22 issued 9/8/70
23.	Public Document Room	Environmental Conservation Library Minneapolis Public Library 300 Nicollet Mall Minneapolis, Minnesota 55401

INSPECTION STATUS

ENFORCEMENT STATUS

TECHNICAL SPECIFICATION SECTION 6.5.C.3 REQUIRES PROCEDURES BE FOLLOWED FOR MAINTENANCE PERFORMED ON SYSTEMS THAT MAY AFFECT SAFETY. 4ACD3.6.3, "WORK REQUEST AUTHORIZATION CANCELLATION" REQUIRES SYSTEMS AND EQUIPMENT TO BE RETURNED TO NORMAL IF WORK IS CANCELLED. CONTRARY TO THE ABOVE, THE CRD HYDRAULIC SYSTEM WAS NOT RETURNED TO NORMAL IN THAT IT WAS FOUND ON AUGUST 14, 1980 THAT THE SUPPORT CLAMPS WERE NOT INSTALLED ON THE FILTERS AFTER WORK WAS CANCELLED.
(8016 DEFICIENCY)

CONTRARY TO THE SECURITY PLAN ACCESS TO A VITAL AREA WAS NOT PROPERLY CONTROLLED.
(8016 INFRACTION)

CONTRARY TO THE SECURITY PLAN A VITAL AREA KEY WAS NOT PROPERLY CONTROLLED.
(8016 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

OPERATING ROUTINELY.

*LAST IE SITE INSPECTION DATE: SEPTEMBER 18-19, 1980

* INSPECTION REPORT NO: 80-17

SUMMARY

INSPECTION SUMMARY

* INSPECTION ON AUGUST 1-31, (80-16): ROUTINE RESIDENT INSPECTION OF LOGS AND RECORDS, MAINTENANCE PROCEDURES, SURVEILLANCE, PHYSICAL SECURITY, HOUSEKEEPING, RADIOACTIVE WASTE SHIPMENTS AND REVIEW OF IE BULLETINS. THE INSPECTION INVOLVED 108 INSPECTOR-HOURS ONSITE BY TWO NRC INSPECTORS INCLUDING 22 INSPECTOR-HOURS ONSITE DURING OFF-SHIFTS. OF THE EIGHT AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN SIX AREAS; TWO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED IN ONE AREA (FAILURE TO CONTROL ACCESS TO PROTECTED AREAS); AND ONE ITEM OF NONCOMPLIANCE IN ANOTHER AREA (FAILURE TO FOLLOW MAINTENANCE PROCEDURES).

* INSPECTION ON SEPTEMBER 15-19, (80-17): ROUTINE, UNANNOUNCED INSPECTION OF CORE POWER DISTRIBUTION LIMITS, REACTOR SHUTDOWN MARGIN DETERMINATION, REACTIVITY ANOMALY DETERMINATION, AND CONTROL ROD SCRAM TIME TESTS. THE INSPECTION INVOLVED 19 INSPECTOR-HOURS ONSITE BY TWO NRC INSPECTORS INCLUDING ZERO INSPECTOR-HOURS ONSITE DURING OFF-SHIFTS. NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE.			

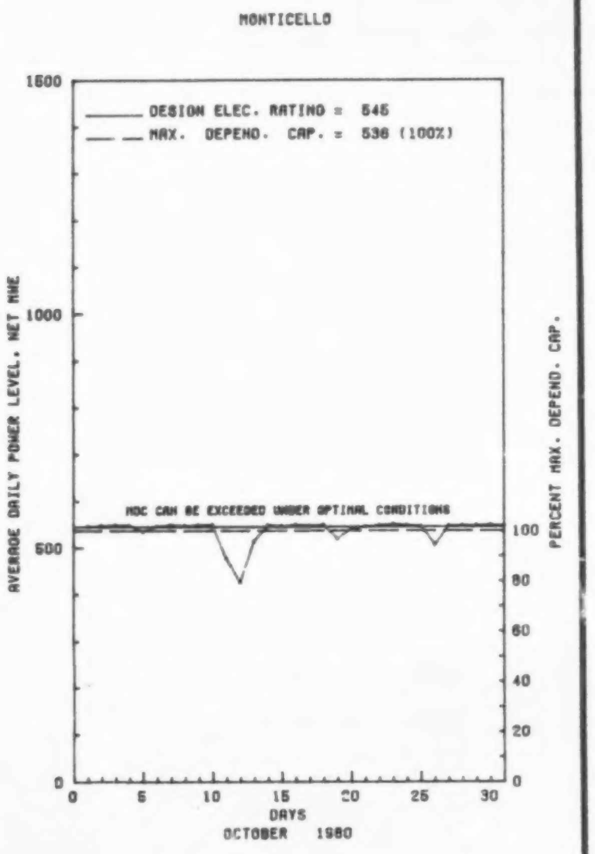
==

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 12-10-80

REVIEWED BY NRR DL: Kenneth T. Eccleston **DATE:** 12-11-80

MONTICELLO

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-263
2. Utility Contact: A. L. Myrabo (612) 295-5151
3. Licensed Thermal Power (MWt): 1670
4. Nameplate Rating (Gross MWe): 632 x 0.9 = 569
5. Design Electrical Rating (Net MWe): 545
6. Maximum Dependable Capacity (Gross MWe): 564
7. Maximum Dependable Capacity (Net MWe): 536
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	745.0	7,320.0	81,865.0
12. Hours Reactor Was Critical	745.0	5,705.2	67,236.1
13. Reactor Reserve Shutdown Hours	.0	.0	940.7
14. Hours Generator On-Line	745.0	5,607.9	65,578.0
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,215,948	8,626,579	107,569,441
17. Gross Electrical Energy (MWH)	415,146	2,914,865	33,892,925
18. Net Electrical Energy (MWH)	399,930	2,779,145	32,411,349
19. Unit Service Factor	100.0	76.6	80.1
20. Unit Availability Factor	100.0	76.6	80.1
21. Unit Capacity Factor (MDC Net)	100.2	70.8	73.9
22. Unit Capacity Factor (DER Net)	98.5	69.7	72.6
23. Unit Forced Outage Rate	.0	9.1	6.7

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
MAY 1, 1981 - REFUELING OUTAGE - 56 DAYS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
NONE									

SUMMARY

MONTICELLO OPERATED AT OR NEAR FULL POWER FOR OCTOBER, WITH ONE REDUCTION ON THE 11TH FOR ALTERATION OF THE CONTROL ROD SEQUENCE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

NINE MILE POINT 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	New York/Oswego 8 miles NE of Oswego
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	1850
5.	Design Electrical Rating (MWE Net)	620
6.	Date of Initial Criticality	September 5, 1969
7.	Date of Commercial Operation	December 1969
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Lake Ontario

Utility & Contractor Information

10.	Licensee	Niagara Mohawk Power Corporation
11.	Corporate Address	300 Erie Boulevard West Syracuse, New York 13202
12.	Corporate Contact	R. Schneider, Vice President Electric Operations
13.	Architect/Engineer	Utility
14.	Nuclear Steam Supply System	GE
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Section Chief	H. B. KISTER
19.	IE Resident Inspector	W. BAUNACK (315) 342-4041 FTS: 264-3311 · Operator
20.	Licensing Project Manager	P. Polk
21.	Docket Number	50-220
22.	License Number/Date of Issue	DPR-63, 12/26/74
23.	Public Document Room	State University College of Oswego Penfield Library · Documents Oswego, NY 13126 (315)341-2323

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO TECHNICAL SPECIFICATIONS AND STANDING ORDERS, THE REMOVABLE SEAL ON THE OUTER REACTOR BUILDING TRACK BAY DOOR WAS NOT PROPERLY INSTALLED WHILE THE INNER DOOR WAS OPEN. THIS RESULTED IN A REDUCTION OF THE SECONDARY CONTAINMENT CAPABILITY.
(8008 INFRACTION)

CONTRARY TO TECHNICAL SPECIFICATIONS AND OPERATING PROCEDURES, VALVES FW-70 AND FW-71 WERE OPEN VICE LOCKED OPEN AS REQUIRED.
(8008 INFRACTION)

CONTRARY TO TECHNICAL SPECIFICATIONS AND ADMINISTRATIVE PROCEDURES, VARIOUS LEADS FROM LPRM DETECTORS WERE LIFTED WITHOUT AN APPROVED PROCEDURE AUTHORIZING SUCH ACTION.
(8008 DEFICIENCY)

CONTRARY TO TECHNICAL SPECIFICATIONS AND ADMINISTRATIVE POLICY, THE FIRE DOOR BETWEEN EMERGENCY DIESEL GENERATOR #102 AND #103 WAS BLOCKED OPEN.
(8008 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS:

- THE PLANT IS OPERATING AT 97% POWER DUE TO END OF CYCLE DERATING ON FOUR RECIRC PUMPS. REPRESENTATIVE STACK AND OFF-GAS RELEASE RATES ARE 10 AND 13,000 MICROCURIES PER SECOND, RESPECTIVELY.
- LAST IE SITE INSPECTION DATE: 10/27-31/80
- INSPECTION REPORT NO: 50-220/80-18

SUMMARY

INSPECTION SUMMARY

- 50-220/80-08 - AUG 16 - SEP 15: ROUTINE, ONSITE REGULAR AND BACKSHIFT INSPECTIONS BY THE RESIDENT INSPECTORS (66 HRS). AREAS INSPECTED INCLUDED OPERATIONAL SAFETY VERIFICATION, WITNESSING OF SURVEILLANCE TESTS, OBSERVATION OF PHYSICAL SECURITY, PLANT TOURS, EQUIPMENT CONTROLS, JUMPER/LIFTED LEADS CONTROLS, I&E BULLETINS AND CIRCULARS, AND AN NRR MEETING WITH THE LICENSEE ON THE REACTOR BUILDING CRACK. FOUR NONCOMPLIANCES WERE IDENTIFIED: FAILURE TO MAINTAIN REACTOR BUILDING TRACK BAY DOOR SEALED; BLOCKING OPEN A FIRE DOOR; FAILURE TO MAINTAIN VALVES LOCKED; FAILURE TO MEET PROCEDURAL REQUIREMENTS FOR LIFTING OF LEADS.
- 50-220/80-09 - SEP 8-11: ROUTINE, ONSITE, UNANNOUNCED INSPECTION BY ONE REGION-BASED INSPECTOR (26 HRS) OF TORUS MODIFICATION ACTIVITIES INCLUDING REVIEW OF PROCEDURES AND RECORDS AND OBSERVATIONS OF WORK IN PROGRESS. NO NONCOMPLIANCES WERE IDENTIFIED.
- 50-220/80-12 - SEP 22-26: ROUTINE, UNANNOUNCED PHYSICAL SECURITY INSPECTION BY ONE REGION-BASED INSPECTOR AND A HQ SECURITY SPECIALIST (72 HRS) OF THE IMPLEMENTATION OF THE APPROVED PHYSICAL SECURITY PLAN RELATIVE TO SECURITY PROGRAM AUDIT, SECURITY ORGANIZATION, PHYSICAL BARRIERS (PA/VA), LIGHTING, ACCESS CONTROL (PA/VA), ALARM STATIONS, DETECTION AIDS, ASSESSMENT AIDS, COMMUNICATIONS, TESTING AND MAINTENANCE, COMPENSATORY MEASURES, RESPONSE, LOCKS, KEYS AND COMBINATIONS AND RECORDS AND REPORTS. FOLLOWUP ON PREVIOUS ITEMS OF NONCOMPLIANCE RELATED TO PHYSICAL SECURITY. NO NONCOMPLIANCES WERE IDENTIFIED.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

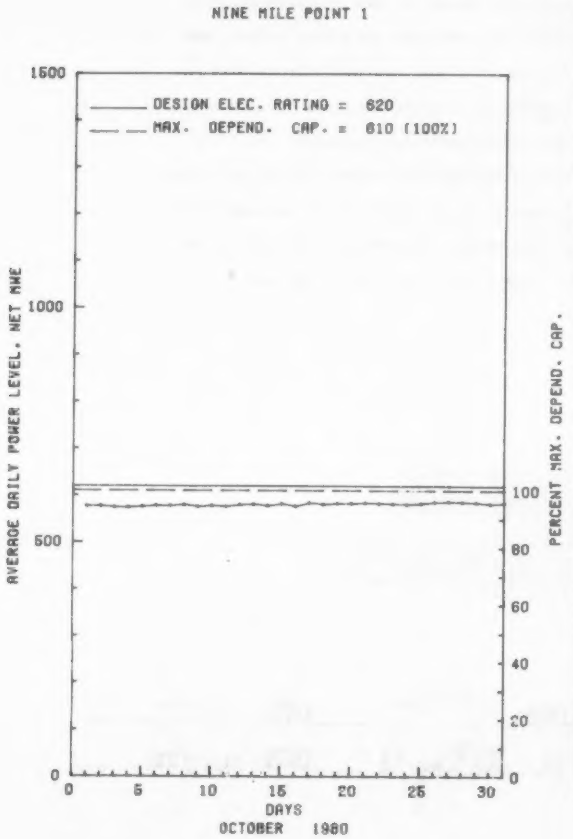
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-013/ 01T	07/15/80	07/25/80	CRD FAILED TO SCRAM DURING MANUAL SCRAM TEST
80-016/ 03L	08/08/80	09/05/80	TURBINE ANTICIPATORY TRIP-BYPASS REACTOR SCRAM SWITCHES FOUND OUT OF TOLERANCES DURING SURVEILLANCE TESTING
80-017/ 03L	08/22/80	09/19/80	REACTOR FEEDWATER PUMP TAKEN OUT OF SERVICE DUE TO SEAL LEAKAGE. REPAIRS WERE COMPLETED WITHIN TIME PERMITTED BY TECHNICAL SPECIFICATIONS.
80-018/ 03L	08/08/80	09/05/80	DIESEL GENERATOR FUEL OIL DAY TANK ACCIDENTALLY DRAINED.
80-019/ 04T	08/27/80	09/09/80	FAILURE TO INSTALL SEAL ON RX BLDG TRACK BAY DOOR.
80-020/ 03L	08/30/80	09/29/80	LOSS OF METEROLOGICAL DATA TO CONTROL ROOM INSTRUMENT
80-021/ 03L	09/18/80	10/08/80	FAILURE TO S.O.R.C. REVIEW PROCEDURE CHANGE IN 7 DAYS.

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 12-10-80

REVIEWED BY NRR DL: Philip J. Polk **DATE:** 12/11/80

NINE MILE POINT 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-220
2. Utility Contact: J. HALLENBECK (315) 343-2110 X1312
3. Licensed Thermal Power (MWt): 1850
4. Nameplate Rating (Gross MWe): 755 x 0.85 = 642
5. Design Electrical Rating (Net MWe): 620
6. Maximum Dependable Capacity (Gross MWe): 630
7. Maximum Dependable Capacity (Net MWe): 610
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): 579
10. Reasons for Restrictions, If Any: EOC DERATE TO 95%

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	745.0	7,320.0	96,432.0
12. Hours Reactor Was Critical	745.0	6,772.7	72,125.4
13. Reactor Reserve Shutdown Hours	.0	.0	1,204.2
14. Hours Generator On-Line	745.0	6,671.1	69,481.6
15. Unit Reserve Shutdown Hours	.0	.0	20.2
16. Gross Thermal Energy (MWH)	1,340,504	11,713,884	113,455,283
17. Gross Electrical Energy (MWH)	446,686	3,883,586	37,393,096
18. Net Electrical Energy (MWH)	432,726	3,758,388	36,208,153
19. Unit Service Factor	100.0	91.1	72.1
20. Unit Availability Factor	100.0	91.1	72.1
21. Unit Capacity Factor (MDC Net)	95.2	84.2	61.6
22. Unit Capacity Factor (DER Net)	93.7	82.8	60.6
23. Unit Forced Outage Rate	.0	5.7	9.0

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUELING SCHEDULED FOR 3/1/81; DURATION: 2 MONTHS
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
NONE									

SUMMARY

NINE MILE POINT 1 OPERATED NORMALLY FOR OCTOBER WITH NO OUTAGES OR REDUCTIONS REPORTED.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

NORTH ANNA 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Louisa County, Virginia 40 Miles NW of Richmond, Virginia
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	PWR
4.	Licenses Power Level (MWT)	2775
5.	Design Electrical Rating (MWE Net)	907
6.	Date of Initial Criticality	April 5, 1978
7.	Date of Commercial Operation	June 6, 1978
8.	Condenser Cooling Method	Once through
9.	Condenser Cooling Water Source	Lake Anne

Utility & Contractor Information

10.	Licensee	Virginia Electric & Power Co.
11.	Corporate Address	P. O. Box 26666 Richmond, Virginia 23261
12.	Corporate Contact	J. H. Ferguson Exec. V.P. - Power
13.	Architect/Engineer	Stone & Webster
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. Lewis
18.	IE Section Chief	P. J. Kellog
19.	IE Resident Inspector	E. H. Webster (703) 894-5421
20.	Licensing Project Manager	L. Engle
21.	Docket Number	50-338
22.	License Number/Date of Issue	NPF-4, 11/26/77
23.	Public Document Room	Alderman Library/Manuscripts Dept. Univ. of Virginia/Charlottesville VA 22901 & Louisa County Courthouse, Louisa, VA 23093

INSPECTION STATUS

ENFORCEMENT STATUS

AS REQUIRED BY UNIT 1 TECHNICAL SPECIFICATION 6.8.1C, WRITTEN PROCEDURES SHALL BE ESTABLISHED, IMPLEMENTED AND MAINTAINED FOR CONDUCT OF SURVEILLANCE AND TEST ACTIVITIES OF SAFETY RELATED EQUIPMENT. CONTRARY TO THE ABOVE, UNIT 1 PERIODIC TEST PROCEDURE 1-PT-36.10, "P-4 PERMISSIVE VERIFICATION," WRITTEN TO VERIFY PROPER OPERATION OF REACTOR TRIP BREAKER AUXILIARY CONTACTS FEEDING P-4 EVERY 31 DAYS, WAS NOT PERFORMED BETWEEN THE DATES OF JANUARY 14, 1980 AND MAY 8, 1980.
(8019 INFRACTION)

VEPCO LETTER SERIAL NO. 1160 DATED DECEMBER 28, 1979, A SUPPLEMENT TO UNIT 1 LER 79-149, STATED THAT STATION EMERGENCY PROCEDURES WOULD BE REVISED AS NECESSARY TO ASSURE CONTINUED VENTILATION FLOW FROM THE SAFEGUARDS BUILDINGS AND THE AUXILIARY BUILDING CENTRAL AREA IN THE EVENT OF DUCTWORK FAILURE DURING A SEISMIC EVENT. CONTRARY TO THE ABOVE, UNITS 1 AND 2 ABNORMAL OPERATING PROCEDURE 1-AP-36, "SEISMIC EVENT," WAS REVISED MARCH 5, 1980, BUT INSTRUCTIONS WERE NOT PROVIDED FOR ASSURING CONTINUED VENTILATION OF THE SAFEGUARDS BUILDINGS IN THE EVENT OF FAILURE OF EXHAUST DUCTWORK.
(8019 DEVIATION)

VEPCO LETTER SERIAL NO. 1179 DATED DECEMBER 12, 1979, CONCERNING COMMITMENTS TO CORRECT IDENTIFIED EQUIPMENT PROBLEMS, STATED THAT ANY USE OF SERVICE WATER MODE OF CONTAINMENT COOLING WOULD HAVE TO BE UNDER STRICT ADMINISTRATIVE CONTROL AND THESE CONTROLS FOR UNIT 1 WOULD BE IN PLACE PRIOR TO STARTUP FROM THE FIRST REFUELING OUTAGE. CONTRARY TO THE ABOVE, NO ADMINISTRATIVE CONTROLS WERE IMPLEMENTED UNTIL AFTER MAY 16, 1980. STARTUP FROM THE FIRST REFUELING WAS IN JANUARY 1980.
(8019 DEVIATION)

AS REQUIRED BY CRITERION XVI OF APPENDIX B TO 10 CFR 50, AND IMPLEMENTED BY VEPCO TOPICAL REPORT VEP-1-3A, SECTION 17, PARAGRAPH 17.2.16, STATES IN PART THAT SIGNIFICANT ADVERSE QUALITY CONDITIONS ARE IDENTIFIED, DOCUMENTED, THEIR CAUSE DETERMINED, AND CORRECTIVE ACTIONS TAKEN TO PRECLUDE THEIR REPETITION. VEPCO'S RESPONSE TO A NOTICE OF VIOLATION 338/79-47-01, DATED JANUARY 15, 1980, DESCRIBED THE CORRECTIVE ACTIONS AND SPECIFIED MARCH 15, 1980 AS THE DATE WHEN FULL COMPLIANCE WOULD BE ACHIEVED. CONTRARY TO THE ABOVE, ON SEPTEMBER 11, 1980, THE INSPECTOR DETERMINED BY RECORD REVIEW AND INTERVIEWS THAT FULL COMPLIANCE HAD NOT BEEN ACHIEVED AS THE SPECIFIED CORRECTIVE ACTIONS HAD NOT TAKEN PLACE.
(8033 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

FULL POWER OPERATION

◆ LAST IE SITE INSPECTION DATE: SEPTEMBER 12, 1980

◆ INSPECTION REPORT NO: 50-338/80-33

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-061/ 01T-0	07/08/80	08/27/80	EMERGENCY FEED PUMP MAY NOT BE ABLE TO DELIVER DESIGN FLOW FOR CERTAIN ACCIDENT CONDITIONS.
80-065/ 03L-0	08/07/80	09/05/80	PRESSURIZER PROTECTION CHANNEL III LEVEL INDICATION 6% HIGHER THAN CHANNELS I&II.
80-068/ 03L-0	08/01/80	08/27/80	ELECTRIC LOAD ADDED TO 1H EMERGENCY BUS WITHOUT APPROVED JUMPER.
80-070/ 03L-0	08/14/80	08/27/80	CONTAINMENT AIR PARTICULATE HIGH FLOW ALARM ACTIVATED WHEN RELAY DISCONNECTED.
80-071/ 01T-0	08/18/80	08/29/80	INSUFFICIENT INFORMATION TO DEMONSTRATE LIT-RS-151A, B AND 251A, B PER NUREG-0588.
80-072/ 03L-0	08/08/80	08/27/80	CONTAINMENT BULK AVERAGE TEMPERATURE EXCEEDED TS LIMIT OF 105 DEGREES.
80-073/ 03L-0	08/18/80	09/16/80	ONE LHSI SYSTEM WAS COMPROMISED BY DE-ENERGIZING MOV-186 A VALVE.
80-074/ 03L-0	08/10/80	09/03/80	AXIAL FLUX DIFFERENCE DEVIATED GREATER THAN 5% AND QUADRANT TILT EXCEEDED 1.02%.
80-076/ 03L-0	08/20/80	09/16/80	ACCEPTANCE CRITERIA NOT MET FOR SURVEILLANCE OF 1-RS-P-3A PUMP.
80-077/ 03L-0	08/21/80	09/18/80	MISALIGNED LHSI PUMP MINIMUM FLOW RECIRCULATION VALVES.

SUMMARY

INSPECTION SUMMARY

◆ INSPECTION JULY 12 - AUGUST 20 (80-30): THIS ROUTINE INSPECTION BY THE RESIDENT INSPECTORS INVOLVED 88 HOURS ON SITE IN THE AREAS OF SURVEILLANCE ACTIVITIES, IE BULLETINS, OPERATIONAL SAFETY AND LICENSEE EVENT REPORTS. OF THE FOUR AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

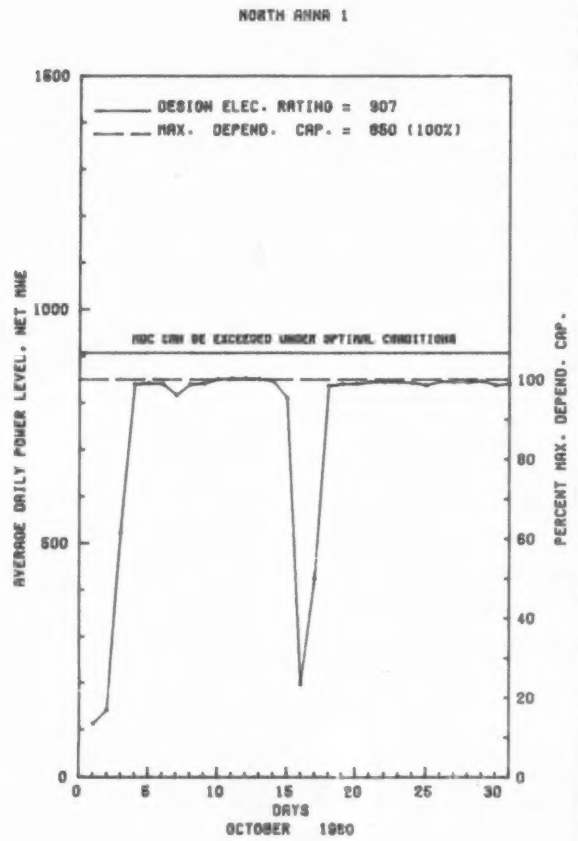
INSPECTION SEPTEMBER 11-12 (80-33): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 6 INSPECTOR-HOURS ONSITE IN THE AREAS OF LICENSEE ACTIONS ON PREVIOUS INSPECTION FINDINGS AND VISUAL WELD EXAMINATION PROCEDURE. OF THE TWO AREAS INSPECTED, ONE ITEM OF NONCOMPLIANCE WAS IDENTIFIED IN ONE AREA (INFRACTION - FAILURE TO TAKE CORRECTIVE ACTION).

REVIEWED BY OMPA LOEB: R Bobe **DATE:** 12/10/80

REVIEWED BY NRR DL: LB Engle **DATE:** 12/11/80

NORTH ANNA 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-338
2. Utility Contact: W. R. Madison (703) 894-5151
3. Licensed Thermal Power (MWt): 2775
4. Nameplate Rating (Gross MWe): 1089 x 0.9 = 980
5. Design Electrical Rating (Net MWe): 907
6. Maximum Dependable Capacity (Gross MWe): 880
7. Maximum Dependable Capacity (Net MWe): 850
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	745.0	7,320.0	21,097.0
12. Hours Reactor Was Critical	741.9	6,435.6	16,600.8
13. Reactor Reserve Shutdown Hours	3.1	64.8	213.1
14. Hours Generator On-Line	729.1	6,223.2	16,276.3
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,843,946	15,889,288	42,169,201
17. Gross Electrical Energy (MWH)	590,995	5,002,563	13,349,012
18. Net Electrical Energy (MWH)	559,612	4,707,300	12,560,746
19. Unit Service Factor	97.9	85.0	77.1
20. Unit Availability Factor	97.9	85.0	77.1
21. Unit Capacity Factor (MDC Net)	88.4	75.7	70.0
22. Unit Capacity Factor (DER Net)	82.8	70.9	65.6
23. Unit Forced Outage Rate	2.1	8.3	9.7

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUEL., TMI & FIRE PROTECTION MODS. - 12/31/81 - 6 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-23	09/30/80	F	6.2	A	4				CONTINUATION OF OUTAGE DUE TO TUBE FAILURE.
80-24	10/02/80	F	9.7	A	9				UNABLE TO OPERATE DUE TO 6TH POINT HEATER TUBE FAILURE. RAMPED UNIT OFF THE LINE, KEPT REACTOR CRITICAL. REPAIRED FAILED TUBES.
80-25	10/15/80	F	.0	H	5				SIGNIFICANT POWER REDUCTION GREATER THAN 20% OF THE AVERAGE DAILY POWER LEVEL.

SUMMARY

NORTH ANNA 1 OPERATED AT NEAR FULL POWER FOR OCTOBER, WITH TWO OUTAGES FOR EQUIPMENT FAILURE ON THE 1ST AND 2ND, AND ONE REDUCTION ON THE 15TH.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

best copy available

NORTH ANNA 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Louisa County, VA 40 Miles NW of Richmond, VA
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	PWR
4.	Licensed Power Level (MWT)	2775
5.	Design Electrical Rating (MWE Net)	907
6.	Date of Initial Criticality	June 12, 1980
7.	Date of Commercial Operation	N/A
8.	Condenser Cooling Method	Once through
9.	Condenser Cooling Water Source	Lake Anna

Utility and Contractor Information

10.	Licensee	Virginia Electric & Power Co.
11.	Corporate Address	P.O. Box 26666 Richmond, VA 23261
12.	Corporate Contact	J. H. Ferguson, Exec. V.P. - Power
13.	Architect/Engineer	Stone & Webster
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. Lewis
18.	IE Section Chief	P. J. Kellog
19.	IE Resident Inspector	E. H. Webster (703) 894-5421
20.	Licensing Project Manager	L. Engle
21.	Docket Number	50-339
22.	License Number/Date of Issue	NPF-7, 8/21/80
23.	Public Document Room	Alderman Library/Manuscripts Dept. Univ. of Virginia/Charlottesville, VA 22901 & Louisa County Courthouse, Louisa, VA 23093

INSPECTION STATUS

ENFORCEMENT STATUS

VEPCO'S "TOPICAL REPORT - QUALITY ASSURANCE PROGRAM - OPERATIONS PHASE" (VEP-3A), SECTION 17.2.27, AS IMPLEMENTED BY PARAGRAPH 2.5.3.2 OF THE "NUCLEAR POWER STATION QUALITY ASSURANCE MANUAL", REQUIRES THAT TEMPORARY PERSONNEL BE INDOCTRINATED IN VEPCO'S QUALITY ASSURANCE PROCEDURES AS RELATED TO THEIR SPECIFIC TASKS TO BE PERFORMED. CONTRARY TO THE ABOVE, THE BABCOCK AND WILCOX STARTUP TEST ENGINEER IN CHARGE OF UNIT 2 PRECRITICAL TEST 2-SW-11 ON THE 4:00 P.M. MIDNIGHT SHIFT ON MAY 1, 1980, HAD NOT RECEIVED SUCH INDOCTRINATION.
(8020 DEVIATION)

AS REQUIRED BY CRITERION V OF APPENDIX B OF 10 CFR 50 AS IMPLEMENTED BY PARAGRAPH 17.2.5 OF VEPCO'S "TOPICAL REPORT - QUALITY ASSURANCE PROGRAM - OPERATIONS PHASE" (VEP-3A) REQUIRES THAT ACTIVITIES AFFECTING QUALITY BE PERFORMED IN ACCORDANCE WITH PROCEDURES. CONTRARY TO THE ABOVE, REWORK OF THE UNIT 2 CONTROL ROOM AIR CONDITIONING CHILLERS AND THEIR CONTROL PANELS PER ENGINEERING AND DESIGN COORDINATION REPORT P-2717-2 WAS ACCOMPLISHED WITHOUT THE USE OF A REWORK CONTROL FORM. USE OF THE REWORK CONTROL FORM IS REQUIRED BY CONSTRUCTION QUALITY CONTROL PROCEDURE 26.1 "REWORK CONTROL PROGRAM."
(8020 INFRACTION)

VEPCO LETTER SERIAL NO. 1160 DATED DECEMBER 28, 1979, A SUPPLEMENT TO UNIT 1 LER 79-149, STATED THAT STATION EMERGENCY PROCEDURES WOULD BE REVISED AS NECESSARY TO ASSURE CONTINUED VENTILATION FLOW FROM THE SAFEGUARDS BUILDINGS AND THE AUXILIARY BUILDING CENTRAL AREA IN THE EVENT OF DUCTWORK FAILURE DURING A SEISMIC EVENT. CONTRARY TO THE ABOVE, UNITS 1 AND 2 ABNORMAL OPERATING PROCEDURE 1-AP-36, "SEISMIC EVENT," WAS REVISED MARCH 5, 1980, BUT INSTRUCTIONS WERE NOT PROVIDED FOR ASSURING CONTINUED VENTILATION OF THE SAFEGUARDS BUILDINGS IN THE EVENT OF FAILURE OF EXHAUST DUCTWORK.
(8020 DEVIATION)

VEPCO LETTER SERIAL NO. 123 OF FEBRUARY 14, 1980, A FINAL REPORT PER 10 CFR 50.55(E) ON THE NON-SEISMICALLY QUALIFIED CONTROL ROOM AIR CONDITIONING CHILLER CONTROL PANELS, STATED THAT REQUIRED UNIT 2 MODIFICATION WOULD BE COMPLETED PRIOR TO INITIAL FUEL LOADING. CONTRARY TO THE ABOVE, ELECTRICAL MODIFICATIONS PER ENGINEERING AND DESIGN COORDINATION REPORT P-2717-2 WERE NOT COMPLETED UNTIL APRIL 28, 1980. INITIAL FUEL LOADING BEGAN APRIL 12, 1980.
(8020 DEVIATION)

THE FIRE PROTECTION SYSTEM REVIEW, SUPPLEMENT NUMBER 1, DATED DECEMBER 15, 1977, IN RESPONSE TO REQUEST 15B THE LICENSEE COMMITTED TO HAVING FOUR PORTABLE RADIOS AVAILABLE FOR USE IN FIGHTING FIRES IN THE CONTROL ROOM AREA. CONTRARY TO THE ABOVE, ON AUGUST 1, 1980, DURING A FIRE DRILL, NO PORTABLE RADIOS WERE AVAILABLE FOR THE FIRE TEAM IN THE CONTROL ROOM.
(8029 DEVIATION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

CONTINUE POWER ESCALATION TESTING

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

ABOUT 75% POWER

+ LAST IE SITE INSPECTION DATE: AUGUST 20, 1980

+ INSPECTION REPORT NO: 50-339/80-29

SUMMARY

INSPECTION SUMMARY

INSPECTION JULY 12 - AUGUST 20 (80-29): THIS ROUTINE INSPECTION BY THE RESIDENT INSPECTORS INVOLVED 186 HOURS ON SITE IN THE AREAS OF NEAR TERM OPERATING LICENSE FOLLOWUP, EMERGENCY PLANNING, LOW POWER PHYSICS TESTING, IE BULLETINS, FIRE DRILL CONDUCT, AND OPERATIONAL SAFETY. OF THE SIX AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN FIVE AREAS. ONE APPARENT DEVIATION WAS IDENTIFIED IN ONE AREA (DEVIATION - FAILURE TO MAINTAIN PORTABLE RADIOS FOR FIRE BRIGADE USE AS COMMITTED TO IN THE FIRE PROTECTION SYSTEM REVIEW).

REPORTS RECEIVED FROM LICENSEE

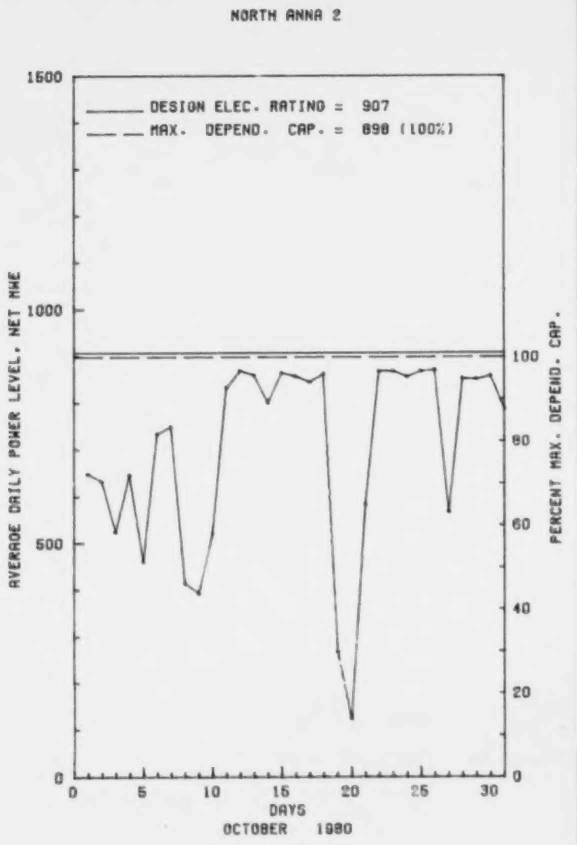
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-037/ 03L-0	08/07/80	09/05/80	EXCEEDED BORON CONCENTRATION IN A AND C SI ACCUMULATORS.
80-043/ 03L-0	08/14/80	08/27/80	PRESSURIZER POWER OPERATED RELIEF VALVE DECLARED INOPERABLE.
80-044/ 03L-0	07/30/80	08/27/80	AT 3% POWER COOL DOWN OF RCS TO A TAVE OF 537 DEGREES.
80-045/ 03L-0	08/03/80	08/27/80	ROD INDICATION N-9 SHUTDOWN BANK A DEVIATED FROM GROUP GREATER THAN 12 STEPS.
80-046/ 03L-0	08/18/80	08/27/80	ROD INDICATION G-9 DEVIATED FROM GROUP GREATER THAN 12 STEPS.
80-047/ 03L-0	08/19/80	09/18/80	AT 3% POWER STEAM FLOW INDICATION CHANNEL 3 EXCEEDED TOLERANCES.
80-048/ 03L-0	08/28/80	09/16/80	STEAM-FEED FLOW MISMATCH TRIP FEED FLOW SQUARE ROOT COMPARATOR CARD OUT OF CALIBRATION.
80-049/ 03L-0	08/22/80	09/16/80	AT 3% POWER CHANNEL C STEAM FLOW, FOUND ISOLATED.
80-050/ 03L-0	09/02/80	10/01/80	PRESSURIZER POWER OPERATED RELIEF VALVE DECLARED INOPERABLE.
80-051/ 03L-0	08/21/80	09/16/80	C FEEDLINE ISOLATION CHECK VALVE, 2-FW-126 DID NOT MEET ACCEPTANCE CRITERIA.
80-052/ 03L-0	08/23/80	09/12/80	C MAIN STEAM TRIP VALVE WOULD NOT FULLY CLOSE DURING TEST.
80-053/ 03L-0	08/25/80	09/16/80	C STEAM FLOW CHANNEL 4 INDICATING LOW.
80-054/ 03L-0	09/09/80	10/01/80	BORIC ACID PUMP PLACED IN SERVICE WITHOUT PERFORMANCE PT-15.2
80-055/ 03L-0	09/05/80	10/01/80	CHECK VALVE 2-SI-70 DID NOT MEET LEAKAGE LIMIT OF TS.
80-056/ 03L-0	09/17/80	10/01/80	PERSONNEL AIR LOCK OUTER DOOR WOULD NOT CLOSE PROPERLY DUE TO MISALIGNMENT.
80-057/ 03L-0	09/08/80	10/01/80	ROD G-9 INDICATION DEVIATED IN SHUTDOWN BANK B BY GREATER THAN TS LIMIT 12 STEPS.
80-060/ 03L-0	09/13/80	10/01/80	PERSONNEL AIR LOCK DID NOT MEET LEAKAGE ACCEPTANCE LIMIT.
80-061/ 03L-0	09/19/80	10/01/80	LHSI PUMP B RELIEF VALVE LIFTED BEFORE DESIGN LIMIT.
80-063/ 03L-0	09/20/80	10/01/80	DISCOVERED ZERO POINT SHIFT IN DELTA T/TAVG CHANNEL 3 AXIAL FLUX SUMMING AMP.
80-071/ 01T-0	08/18/80	08/29/80	INSUFFICIENT INFORMATION TO DEMONSTRATE LIT-RS-151A, B AND 251A, B PER NUREG-0588.

REVIEWED BY OMPA LOEB: P E Robe **DATE:** 12/10/80

REVIEWED BY NRR DL: **DATE:** 12/11/80

NORTH ANNA 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-339
2. Utility Contact: W. A. WOODSMALL, III (703) 894-5151
3. Licensed Thermal Power (MWt): 2775
4. Nameplate Rating (Gross MWe): 947
5. Design Electrical Rating (Net MWe): 907
6. Maximum Dependable Capacity (Gross MWe): 928
7. Maximum Dependable Capacity (Net MWe): 898
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	745.0	3,673.0	3,673.0
12. Hours Reactor Was Critical	722.5	1,935.7	1,935.7
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	714.3	1,228.7	1,228.7
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,685,732	2,345,050	2,345,050
17. Gross Electrical Energy (MWH)	551,312	753,332	753,332
18. Net Electrical Energy (MWH)	522,444	706,553	706,553
19. Unit Service Factor			
20. Unit Availability Factor			
21. Unit Capacity Factor (MDC Net)			
22. Unit Capacity Factor (DER Net)			
23. Unit Forced Outage Rate			

NOT IN COMMERCIAL OPERATION

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
THI & FIRE PROTECT MODS 11/1/80 22 DAYS; MAINT 05/81 11 DAYS
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause and Corrective Action to Prevent Recurrence
80-9	10/19/80	F	25.2	A	3				GENERATOR-TURBINE/REACTOR TRIP WHEN 2A STATION SERVICE TRANSFORMER LOCK OUT RELAY ACTUATED. THE RELAY ACTUATED WHEN A LOW SIDE CABLE, ON THE 2A TRANSFORMER, BLEW OUT. REPAIRED THE BAD CABLE AND RETURNED UNIT TO OPERATION.
80-10	10/20/80	F	3.5	G	3				LO-LEVEL "C" S/G WHILE RESTARTING THE UNIT, OPERATOR EXPERIENCE.
80-11	10/31/80	S	2.0	B	3				A SCHEDULED TRIP OCCURRED WHEN THE GENERATOR OUTPUT BREAKERS WERE OPENED IN ACCORDANCE WITH 2-SU-27 100% LOAD REJECT TEST.

SUMMARY

NORTH ANNA 2 OPERATED NORMALLY DURING OCTOBER, WITH THREE OUTAGES DUE TO EQUIPMENT FAILURE ON THE 19TH, OPERATIONAL ERROR ON THE 20TH, AND TESTING ON THE 31ST.

LEGEND

[1]Type:
F--Forced
S--Scheduled

[2]Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & Licensee Examination
F--Administrative
G--Operational Error
H--Other

[3]Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4]Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5]Exhibit H:
Same source

OCONEE 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	South Carolina/Oconee 30 miles W of Greenville
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2568
5.	Design Electrical Rating (MWE Net)	887
6.	Date of Initial Criticality	April 19, 1973
7.	Date of Commercial Operation	July 15, 1973
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Keowee Lake

Utility & Contractor Information

10.	Licensee	Duke Power Company
11.	Corporate Address	422 S. Church Street Charlotte, North Carolina 28242
12.	Corporate Contact	W. O. Parker, Jr.
13.	Architect/Engineer	DPC and Bechtel (Containment Bldg.)
14.	Nuclear Steam Supply System	B&W
15.	Constructor	DPC

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R.C. Lewis
18.	IE Section Chief	R. MARTIN
19.	IE Resident Inspector	F. Jape (803) 882-6927
20.	Licensing Project Manager	M. B. Fairtile
21.	Docket Number	50-269
22.	License Number/Date of Issue	DPR-38, 2/6/73
23.	Public Document Room	Oconee County Library 201 S. Spring Street Walhalla, South Carolina 29691

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

- 100% POWER.
- LAST IE SITE INSPECTION DATE: OCTOBER 1-31, 1980

INSPECTION REPORT NO: 50-269/80-33

SUMMARY

INSPECTION SUMMARY

- NO REPORTS RELEASED THIS MONTH.

REPORTS RECEIVED FROM LICENSEE

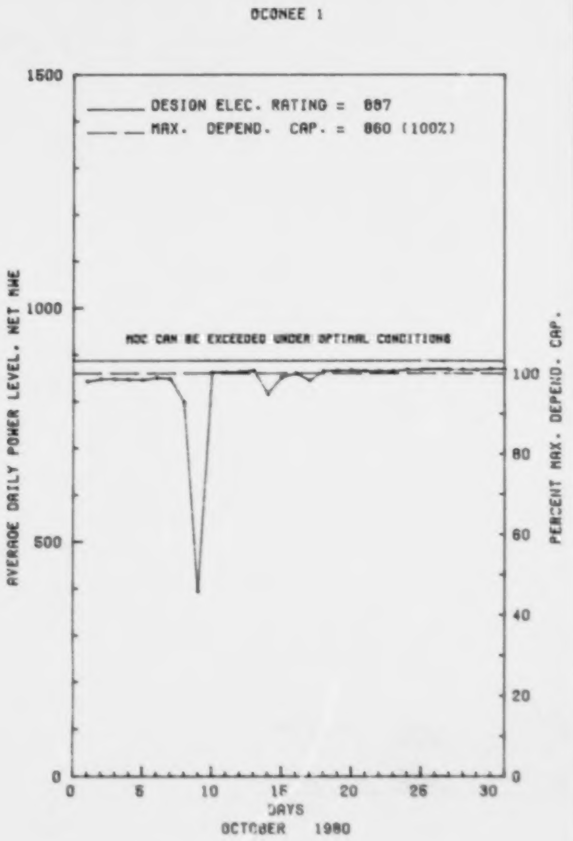
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-015/01T-0	05/23/80	06/06/80	FOUR BROKEN HOLDDOWN SPRINGS DISCOVERED.

REVIEWED BY OMPA LOEB: R Bole **DATE:** 12/10/80

REVIEWED BY NRR DL: M. B. Fairtile **DATE:** 12/10/80

OCONEE 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-269
2. Utility Contact: J. A. Reavis (704) 373-8552
3. Licensed Thermal Power (MWt): 2568
4. Nameplate Rating (Gross MWe): 1038 x 0.9 = 934
5. Design Electrical Rating (Net MWe): 887
6. Maximum Dependable Capacity (Gross MWe): 899
7. Maximum Dependable Capacity (Net MWe): 860

CAPACITY FACTORS CALCULATED WITH A WEIGHTED AVERAGE

8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	745.0	7,320.0	63,961.0
12. Hours Reactor Was Critical	743.5	5,307.5	45,822.2
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	738.8	5,173.2	43,120.3
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,880,721	11,524,500	100,742,619
17. Gross Electrical Energy (MWH)	657,090	4,066,200	34,980,500
18. Net Electrical Energy (MWH)	626,923	3,853,367	33,084,866
19. Unit Service Factor	99.2	70.7	67.4
20. Unit Availability Factor	99.2	70.7	67.4
21. Unit Capacity Factor (MDC Net)	97.8	61.2	59.9
22. Unit Capacity Factor (DER Net)	94.9	59.3	58.3
23. Unit Forced Outage Rate	.8	12.8	17.4

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
10	10/08/80	F	6.2	A	3		HA	TURBIN	TEMPORARY LOSS OF 125 VDC POWER SUPPLY TO THE TURBINE EHC CONTROL CABINET RESULTED IN UNIT TRIP.
11-P	10/14/80	F	.0	A	5		HH	PUMPXX	1B MOISTURE SEPARATOR DRAIN PUMP OUT OF SERVICE.

SUMMARY

OCONEE 1 OPERATED AT NEAR, OR ABOVE FULL POWER IN OCTOBER, WITH ONE OUTAGE AND ONE REDUCTION DUE TO EQUIPMENT FAILURE.

LEGEND

[1] Type:
F–Forced
S–Scheduled

[2] Reason:
A–Equipment Failure
B–Maintenance or Test
C–Refueling
D–Regulatory Restriction
E–Operator Training & License Examination
F–Administrative
G–Operational Error
H–Other

[3] Method:
1–Manual
2–Manual Scram
3–Automatic Scram
4–Continuations
5–Load Reduction
9–Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

OCONEE 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	South Carolina/Oconee 30 miles W of Greenville
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2568
5.	Design Electrical Rating (MWE Net)	887
6.	Date of Initial Criticality	November 11, 1973
7.	Date of Commercial Operation	September 9, 1974
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Keowee Lake

Utility & Contractor Information

10.	Licensee	Duke Power Company
11.	Corporate Address	422 S. Church Street Charlotte, North Carolina 28242
12.	Corporate Contact	W. O. Parker, Jr.
13.	Architect/Engineer	DPC/Bechtel
14.	Nuclear Steam Supply System	B&W
15.	Constructor	DPC

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R.C. Lewis
18.	IE Section Chief	R. MARTIN
19.	IE Resident Inspector	F. Jape (803) 882-6927
20.	Licensing Project Manager	M. B. Fairtile
21.	Docket Number	50-270
22.	License Number/Date of Issue	DPR-47, 10/6/73
23.	Public Document Room	Oconee County Library 201 S. Spring Street Walhalla, South Carolina 29691

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

```
INSPECTION STATUS

OTHER ITEMS OF INTEREST
SYSTEMS AND COMPONENT PROBLEMS:
NONE

FACILITY ITEMS (PLANS AND PROCEDURES):
+   FIVE WEEK MAINTENANCE OUTAGE SCHEDULED TO BEGIN NOVEMBER 7, 1980.
MANAGERIAL ITEMS:
NONE.
PLANT STATUS:
100% POWER.
+   LAST IE SITE INSPECTION DATE:  OCTOBER 1-31, 1980
INSPECTION REPORT NO:  50-270/80-29
SUMMARY
INSPECTION SUMMARY
+   NO REPORTS RELEASED THIS MONTH.
```

REPORTS RECEIVED FROM LICENSEE

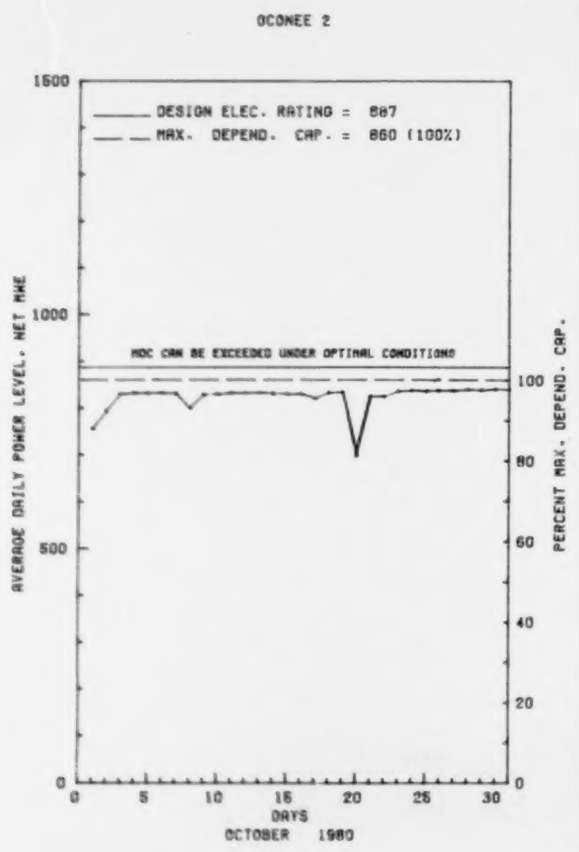
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE.			

REVIEWED BY OMPA LOEB: ______ **DATE:** 12/10/80

REVIEWED BY NRR DL: M. B. Fairtile **DATE:** 12/10/80

OCONEE 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-270
2. Utility Contact: J. A. Reavis (704) 373-8552
3. Licensed Thermal Power (MWt): 2568
4. Nameplate Rating (Gross MWe): 1038 x 0.9 = 934
5. Design Electrical Rating (Net MWe): 887
6. Maximum Dependable Capacity (Gross MWe): 899
7. Maximum Dependable Capacity (Net MWe): 860

CAPACITY FACTORS CALCULATED WITH A WEIGHTED AVERAGE

8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	745.0	7,320.0	53,881.6
12. Hours Reactor Was Critical	745.0	4,707.7	38,303.7
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	745.0	4,605.4	37,381.5
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,886,505	10,881,406	88,128,810
17. Gross Electrical Energy (MWH)	642,440	3,420,650	29,935,006
18. Net Electrical Energy (MWH)	613,597	3,236,275	28,400,033
19. Unit Service Factor	100.0	62.9	69.4
20. Unit Ava..ability Factor	100.0	62.9	69.4
*21. Unit Capacity Factor (MDC Net)	95.8	51.4	61.0
22. Unit Capacity Factor (DER Net)	92.9	49.8	59.4
23. Unit Forced Outage Rate	.0	2.4	18.2

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
TMI-RELATED MODIFICATIONS - NOVEMBER 7, 1980 - 4 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
16-P	10/08/80	F	.0	A	5		HI	VALVEX	STEAM LEAKAGE FROM FDW-104 CAUSED HIGH HUMIDITY IN PENETRATION ROOM. POWER WAS REDUCED PER TECH SPEC. VALVE WAS REPAIRED TEMPORARILY.
17-P	10/20/80	F	.0	A	5		CH	VALVEX	REMOVED FWP 2A FROM SERVICE TO REPAIR FLANGE GASKET LEAK ON PUMP CASING VENT VALVE (2 FDW-281).

SUMMARY

OCONEE 2 OPERATED AT NEAR FULL POWER DURING OCTOBER WITH TWO REDUCTIONS DUE TO EQUIPMENT FAILURE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

OCONEE 3

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	South Carolina/Oconee 30 miles W of Greenville
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3	Type of Reactor	PWR
4.	Power Level (MWT)	2568
5.	Design Electrical Rating (MWE Net)	887
6.	Date of Initial Criticality	September 5, 1974
7.	Date of Commercial Operation	December 16, 1974
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Keowee Lake

Utility & Contractor Information

10.	Licensee	Duke Power Company
11.	Corporate Address	422 S. Church Street Charlotte, North Carolina 28242
12.	Corporate Contact	W. O. Parker, Jr.
13.	Architect/Engineer	DPC/Bechtel
14.	Nuclear Steam Supply System	B&W
15.	Constructor	DPC

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R.C. Lewis
18.	IE Section Chief	R. MARTIN
19.	IE Resident Inspector	F. Jape (803) 882-6927
20.	Licensing Project Manager	M. B. Fairtile
21.	Docket Number	50-287
22.	License Number/Date of Issue	DPR-55, 7/19/74
23.	Public Document Room	Oconee County Library 201 S. Spring Street Walhalla, South Carolina 29691

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

+ REFUELING OUTAGE SCHEDULED TO BEGIN DECEMBER 5, 1980.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

+ 100% POWER.

+ LAST IE SITE INSPECTION DATE: OCTOBER 1-31, 1980

INSPECTION REPORT NO: 50-287/80-26

SUMMARY

INSPECTION SUMMARY

+ NO REPORTS RELEASED THIS MONTH.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

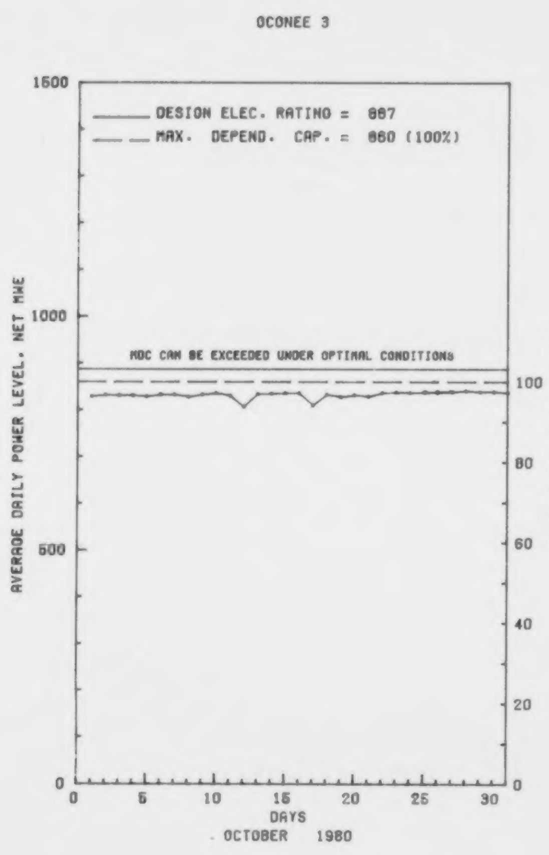
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-012/ 01T-0	09/08/80	09/22/80	BORON ANALYSIS OF THE CONCENTRATED BORIC ACID STORAGE TANK WAS NOT PERFORMED.

REVIEWED BY OMPA LOEB: PE Bobe **DATE:** 12/10/80

REVIEWED BY NRR DL: M.B. Fairtile **DATE:** 12/10/80

OCONEE 3

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-287
2. Utility Contact: J. A. Reavis (704) 373-8552
3. Licensed Thermal Power (MWt): 2568
4. Nameplate Rating (Gross MWe): 1038 x 0.9 = 934
5. Design Electrical Rating (Net MWe): 887
6. Maximum Dependable Capacity (Gross MWe): 899
7. Maximum Dependable Capacity (Net MWe): 860
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

CAPACITY FACTORS CALCULATED WITH A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	745.0	7,320.0	51,528.0
12. Hours Reactor Was Critical	745.0	5,668.1	37,562.0
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	745.0	5,578.6	36,640.5
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,901,356	14,046,696	88,428,301
17. Gross Electrical Energy (MWH)	648,920	4,835,020	30,586,284
18. Net Electrical Energy (MWH)	620,218	4,607,114	29,103,670
19. Unit Service Factor	100.0	76.2	71.1
20. Unit Availability Factor	100.0	76.2	71.1
21. Unit Capacity Factor (MDC Net)	96.8	73.2	65.4
22. Unit Capacity Factor (DER Net)	93.9	71.0	63.7
23. Unit Forced Outage Rate	.0	12.1	17.2

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
REFUELING - DECEMBER 7 - 13 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	NONE								

SUMMARY

OCONEE 3 OPERATED AT NEAR FULL POWER DURING OCTOBER, WITH NO REPORTABLE REDUCTIONS OR OUTAGES.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

OYSTER CREEK

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	New Jersey/Ocean 9 miles S of Toms River
2.	Electric Reliability Council	Mid-Atlantic Area Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	1930
5.	Design Electrical Rating (MWE Net)	650
6.	Date of Initial Criticality	May 3, 1969
7.	Date of Commercial Operation	December 1969
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Barnegat Bay

Utility & Contractor Information

10.	Licensee	Jersey Central Power & Light Co.
11.	Corporate Address	Madison Avenue At Punch Bowl Road Morristown, New Jersey 07960
12.	Corporate Contact	I. Finfrock, Vice President Power Generation
13.	Architect/Engineer	Burns & Roe
14.	Nuclear Steam Supply System	GE
15.	Constructor	J. A. Jones

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	IE Section Chief	R. R. KEIMIG
19.	IE Resident Inspector	L. BRIGGS (X5306) (FTS) 341-3000 (609) 693-0702
20.	Licensing Project Manager	W. A. Paulson
21.	Docket Number	50-219
22.	License Number/Date of Issue	DPR-16, 4/9/69
23.	Public Document Room	Ocean County Library 15 Hooper Avenue Toms River, New Jersey 08753

INSPECTION STATUS

ENFORCEMENT STATUS

10 CFR 50 APPENDIX B, CRITERION II STATES IN PART, "ACTIVITIES AFFECTING QUALITY SHALL BE ACCOMPLISHED UNDER SUITABLY CONTROLLED CONDITIONS. CONTROLLED CONDITIONS INCLUDE THE USE OF APPROPRIATE EQUIPMENT....THE PROGRAM SHALL TAKE INTO ACCOUNT THE NEED FOR SPECIAL CONTROLS...." CONTRARY TO THE ABOVE, A TEMPORARY WASTE WATER PROCESSING DEMINERALIZER WAS INSTALLED WITHOUT ADEQUATE PROCESS CONTROLS OR MONITORING DEVICES TO SAFEGUARD AGAINST THE RELEASE OF RADIOACTIVE MATERIALS TO THE ENVIRONMENT. SUBSEQUENTLY DURING SYSTEM OPERATION, THE DEMINERALIZER OVERFLOWED SPILLING APPROXIMATELY 20 GALLONS OF RADIOACTIVELY CONTAMINATED WATER, ABOUT 2 GALLONS OF WHICH FLOWED OUTSIDE THE CONFINES OF THE BUILDING.
(8028 INFRACTION)

PROCEDURE 915.4, "CONTAMINATION CONTROL," STATES IN PART, "PROTECTIVE CLOTHING IS NOT TO BE WORN OUTSIDE THE CONTROLLED AREA...AT EXITS FROM CONTAMINATED AREAS, AN AREA IS DESIGNATED AS THE STEP OFF PAD. THIS AREA...IS CONSIDERED PART OF THE AREA OUTSIDE THE CONTAMINATED AREA. NOTHING CONTAMINATED SHOULD BE PLACED ON THIS AREA." CONTRARY TO THE ABOVE AN INDIVIDUAL WAS OBSERVED REMOVING POTENTIALLY CONTAMINATED PROTECTIVE CLOTHING IN AREA OUTSIDE THE CONTAMINATED AREA CREATING THE POTENTIAL FOR THE SPREAD OF CONTAMINATION TO UNCONTAMINATED AREAS.
(8028 INFRACTION)

FOUR INDIVIDUALS WERE WORKING IN A CONTROLLED CONTAMINATION AREA PERFORMING DECONTAMINATION ACTIVITIES CONSISTING OF SOAP AND WATER WASHING OF EQUIPMENT AND PIPING. THE INDIVIDUALS WERE WEARING PROTECTIVE CLOTHING LESS THAN THE MINIMUM SPECIFIED IN RADIOACTIVE WORK PERMIT NUMBER 127980. THIS IS IN NONCOMPLIANCE WITH PROCEDURE 915.4, "CONTAMINATION CONTROL."
(8028 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS:

+ PLANT IS OPERATING AT A NOMINAL 97 PERCENT POWER WITH AN OFFGAS RATE OF 82,000 MICROCURIES PER SECOND AND A STACK RELEASE RATE OF 3485 MICROCURIES PER SECOND.

* LAST IE SITE INSPECTION DATE: 10/6-31/80

+ INSPECTION REPORT NO: 50-219/80-32

SUMMARY

INSPECTION SUMMARY

+ 50-219/80-29 - SEP 2 - OCT 1: ROUTINE INSPECTION BY TWO RESIDENT INSPECTORS (48 HRS) OF: FOLLOWUP OF OPERATIONAL EVENTS THAT OCCURRED DURING THE INSPECTION; TOURS OF THE FACILITY; LOG AND RECORD REVIEW; IN-OFFICE LER REVIEW; ON-SITE LER FOLLOWUP; AND, IN-OFFICE REVIEW OF PERIODIC REPORTS. NO NONCOMPLIANCES WERE IDENTIFIED.

+ 50-219/80-31 - SEP 25: SPECIAL, ANNOUNCED MANAGEMENT MEETING TO DISCUSS THE RESULTS OF AN APPRAISAL BOARD CONVENED TO EVALUATE THE PERFORMANCE OF THE LICENSEE FROM AUGUST 1, 1979 TO JULY 31, 1980. AREAS ADDRESSED INCLUDED: PLANT OPERATIONS, REFUELING OPERATIONS, RADITION PROTECTION, RADWASTE OPERATIONS, TRANSPORTATION, MAINTENANCE, SECURITY AND SAFEGUARDS, SURVEILLANCE AND POST REFUEL TESTING, DESIGN CHANGE AND MODIFICATIONS, EMERGENCY PLANNING, ENVIRONMENTAL, TRAINING, MANAGEMENT CONTROLS, FIRE PROTECTION, QA/QC, COMMITTEE ACTIVITIES/AUDITS, REPORTING, AND PROCUREMENT. A SUMMARY OF THE LICENSEE PERFORMANCE EVALUATION WAS PRESENTED. AREAS OF CONCERN WERE DISCUSSED WITH CORPORATE AND SITE MANAGEMENT. NO NEW ENFORCEMENT ACTIONS WERE IDENTIFIED; HOWEVER, A NUMBER OF AREAS WHERE IMPROVEMENTS ARE NECESSARY WERE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

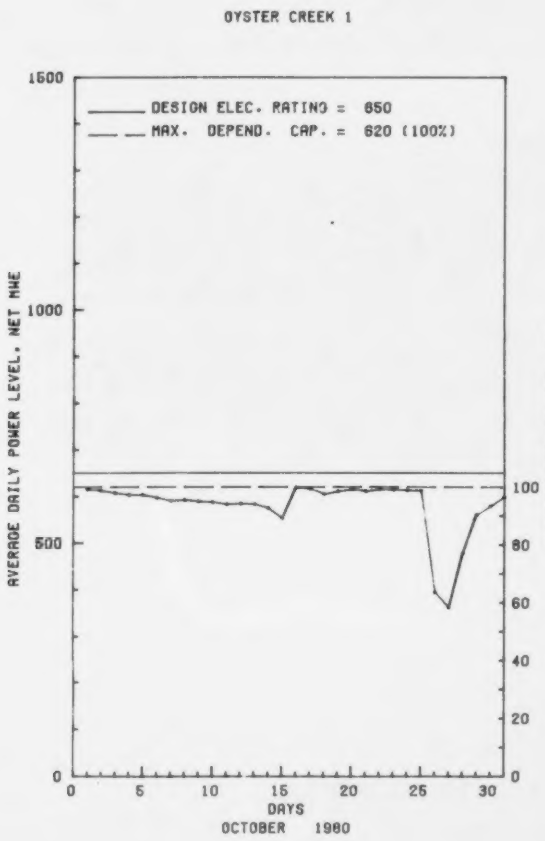
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-035/ 03L	08/14/80	09/10/80	TWO OF FOUR HIGH DRYWELL PRESSURE SWITCHES FOUND .04 AND .15 PSIG ABOVE T.S. LIMIT DURING ROUTINE SURVEILLANCE
80-036/ 03L	08/15/80	09/15/80	STACK RELEASES NOT MONITORED WHEN B SAMPLE PUMP TRIPPED FOR A SHORT PERIOD. EVALUATION OF RECORDER CHARTS READING INPUT TO THE STACK SHOWED NORMAL READINGS DURING THIS TIME PERIOD.
80-037/ 03L	08/19/80	09/18/80	SBGTS NO. 2 FOUND INOPERABLE, SYSTEM NO. 1 TESTED SATISFACTORILY. FAILURE WAS DUE TO WELDING LEAD MISALIGNING A SYSTEM 2 FLOW SENSOR.
80-038/ 03L	08/26/80	09/25/80	ONE OF FOUR REACTOR TRIPLE LOW WATER LEVEL SENSOR SETPOINTS FOUND LESS CONSERVATIVE THAN T.S. LIMIT.
80-039/ 03L	08/29/80	09/25/80	TWO OF OUR MAIN STEAM LINE LOW PRESSURE SENSORS FOUND 5 PSIG BELOW T.S. LIMIT DURING ROUTINE SURVEILLANCE.
80-040/ 03L	09/04/80	09/30/80	ONE HYDRAULIC SNUBBER (CORE SPRAY SYS.I) FAILED TO LOCK-UP IN COMPRESSION OR TENSION DUE TO LOSS OF HYDRAULIC FLUID. FOUND DURING ROUTINE TESTING.
80-041/ 03L	09/05/80	10/01/80	CORE SPRAY SYSTEM I OUT OF SERVICE DUE TO LEAKING PIPE NIPPLE.
80-042/ 03L	09/11/80	10/07/80	THREE OF FOUR DYRWELL HIGH PRESSURE SENSOR SETPOINTS FOUND ABOVE T.S. LIMITS.
80-043/ 03L	09/25/80	10/03/80	ONE OF FOUR TRIPLE LOW SENSOR SETPOINTS FOUND TWO INCHES BELOW ALLOWABLE LIMIT.
80-07E	09/01/80	09/19/80	ONLY ONE DILUTION PUMP IN OPERATION WHEN TWO WERE REQUIRED DUE TO MECHANICAL VIBRATION CAUSING FALSE HIGH LUBE OIL TEMPERATURE TRIPS OF SECOND OPERABLE DILUTION PUMP.
80-08E	09/10/80	09/25/80	ONLY ONE DILUTION PUMP IN OPERATION WHEN TWO WERE REQUIRED. ETS LIMIT EXCEEDED FIVE TIMES IN EIGHT DAY PERIOD DUE TO MECHANICAL FAILURE OF TWO OF THREE DILUTION PUMPS.

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 12-10-80

REVIEWED BY NRR DL: Walter A. Paulson **DATE:** 12-10-80

OYSTER CREEK

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-219
2. Utility Contact: C. M. McClain (201) 455-8748
3. Licensed Thermal Power (MWt): 1930
4. Nameplate Rating (Gross MWe): 687.5 x 0.8 = 550
5. Design Electrical Rating (Net MWe): 650
6. Maximum Dependable Capacity (Gross MWe): 650
7. Maximum Dependable Capacity (Net MWe): 620
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): 620
10. Reasons for Restrictions, If Any:
CONDENSATE DEMIN. DIFFERENTIAL PRESSURE LIMITATIONS.

ITEM 21 CALCULATED BY USING A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	745.0	7,320.0	95,184.0
12. Hours Reactor Was Critical	745.0	2,483.6	71,122.7
13. Reactor Reserve Shutdown Hours	.0	.0	468.2
14. Hours Generator On-Line	745.0	2,384.2	69,629.7
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,354,012	4,015,633	117,894,791
17. Gross Electrical Energy (MWH)	448,060	1,284,690	40,167,005
18. Net Electrical Energy (MWH)	431,387	1,226,148	38,707,161
19. Unit Service Factor	100.0	32.6	73.2
20. Unit Availability Factor	100.0	32.6	73.2
21. Unit Capacity Factor (MDC Net)	93.4	27.0	67.1
22. Unit Capacity Factor (DER Net)	89.1	25.8	62.6
23. Unit Forced Outage Rate	.0	8.5	6.6

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
TMI MODS & REFUELING - SPRING 1981.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
5	10/26/80	F	10.1	B	5		ZZ	ZZZZZZ	FOULING OF TBCCW HEAT EXCHANGER WAS CORRECTED BY CLEANING OPERATIONS.

SUMMARY

OYSTER CREEK 1 OPERATED UNDER UTILITY IMPOSED RESTRICTION FOR OCTOBER, WITH ONE REDUCTION ON THE 26TH FOR MAINTENANCE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

PALISADES

FACILITY DATA

Facility Description

1. Location (state/county) (nearest population center)	Michigan/Van Buren 5 miles S of South Haven
2. Electric Reliability Council	East Central Area Reliability Coordination Agreement
3. Type of Reactor	PWR
4. Power Level (MWT)	2530
5. Design Electrical Rating (MWE Net)	805
6. Date of Initial Criticality	May 24, 1971
7. Date of Commercial Operation	December 31, 1971
8. Condenser Cooling Method	Forced Draft Cooling Towers
9. Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10. Licensee	Consumers Power Company
11. Corporate Address	212 W. Michigan Avenue Jackson, Michigan 49201
12. Corporate Contact	R. B. DeWitt
13. Architect/Engineer	Bechtel
14. Nuclear Steam Supply System	Combustion Engineering
15. Constructor	Bechtel

Regulatory Information

16. IE Region responsible	III
17. IE Branch Chief	R. Heishman
18. IE Section Chief	D.C. Boyd
19. IE Resident Inspector	B. Jorgenson (616) 764-8971
20. Licensing Project Manager	R. D. Silver
21. Docket Number	50-255
22. License Number/Date of Issue	DPR-20, 3/24/71
23. Public Document Room	Kalamazoo Public Library 315 South Rose Street Reference Department Kalamazoo, Michigan 49006

INSPECTION STATUS

ENFORCEMENT STATUS

10 CFR 50.72(A)(6) REQUIRES IN PART, THAT LICENSEES NOTIFY THE NRC OPERATIONS CENTER WITHIN ONE HOUR BY TELEPHONE OF PERSONNEL ERRORS WHICH PREVENT OR COULD PREVENT DURING ACCIDENT CONDITIONS THE FULFILLMENT OF THE SAFETY FUNCTION OF COMPONENTS IMPORTANT TO SAFETY THAT ARE NEEDED TO SHUT DOWN THE REACTOR SAFELY AND MAINTAIN IT IN A SAFE SHUTDOWN CONDITION OR REMOVE RESIDUAL HEAT FOLLOWING REACTOR SHUTDOWN. CONTRARY TO THE ABOVE, CONTAINMENT SUMP ISOLATION VALVE CV 3030 WHICH WAS FOUND MISPOSITIONED DUE TO PERSONNEL ERROR IN THE OPEN POSITION DURING NORMAL OPERATION AT 7:36 A.M. ON JULY 27, 1980, AND WHICH COULD HAVE PREVENTED THE FULFILLMENT OF THE SAFETY FUNCTION OF COMPONENTS IMPORTANT TO SAFETY, WAS NOT REPORTED TO THE NRC UNTIL 29 HOURS AFTER THE VALVE HAD BEEN FOUND OPEN.
(8012 DEFICIENCY)

TECHNICAL SPECIFICATIONS 3.3.1 AND 3.3.2 REQUIRE, DURING POWER OPERATION, THE OPERABILITY OF AT LEAST TWO HIGH PRESSURE AND ONE LOW PRESSURE SAFETY INJECTION PUMPS OR ONE HIGH PRESSURE AND TWO LOW PRESSURE SAFETY INJECTION PUMPS (INCLUDING THOSE VALVES DIRECTLY ASSOCIATED WITH THE PUMPS WHICH ARE REQUIRED TO FUNCTION DURING ACCIDENT CONDITIONS). CONTRARY TO THE ABOVE, THE PLANT OPERATED AT APPROXIMATELY 88% POWER THROUGHOUT THE 36 HOUR INTERVAL FROM 7:36 P.M. ON JULY 25, 1980, TO 7:36 A.M. ON JULY 27, 1980, WITH TWO OF THE THREE HIGH PRESSURE SAFETY INJECTION PUMPS (P-66B AND P-66C) AND ONE OF THE TWO LOW PRESSURE SAFETY INJECTION PUMPS (P-67B) INOPERABLE WITH RESPECT TO THEIR DESIGN FUNCTION DUE TO CONTAINMENT SUMP SUCTION VALVE CV 3030 BEING MISPOSITIONED IN THE OPEN POSITION.
(8012 INFRACTION)

TECHNICAL SPECIFICATION 6.8.1.C REQUIRES WRITTEN PROCEDURES FOR SURVEILLANCE AND TESTING ACTIVITIES ON SAFETY-RELATED EQUIPMENT TO BE IMPLEMENTED. SAFETY-RELATED SURVEILLANCE PROCEDURE QO-2 "RECIRCULATION ACTUATION SYSTEM" HAS, AS A PREREQUISITE, THAT THE PLANT BE IN COLD SHUTDOWN OR REFUELING MODE WHEN THE TEST IS PERFORMED. CONTRARY TO THE ABOVE, ON AUGUST 19, 1980, THE LICENSEE PERFORMED TEST QO-2, WHEN THE PLANT WAS OPERATING AT 88% POWER.
(8012 INFRACTION)

TECHNICAL SPECIFICATION 3.6.1A REQUIRES THAT CONTAINMENT INTEGRITY NOT BE VIOLATED UNLESS THE REACTOR IS IN THE COLD SHUTDOWN CONDITION. CONTRARY TO THE ABOVE, FROM 7:36 P.M. ON JULY 25, 1980 TO 7:36 A.M. ON JULY 27, 1980, THE REACTOR WAS OPERATED IN OTHER THAN THE COLD SHUTDOWN CONDITION WITH CONTAINMENT INTEGRITY VIOLATED. SPECIFICALLY, CONTAINMENT SUMP SUCTION VALVE CV 3030 WAS OPEN AND WOULD HAVE ALLOWED PUMPING A LIMITED QUANTITY OF CONTAMINATED SUMP WATER TO THE SIRW TANK SUBSEQUENT TO A LARGE LOCA.
(8012 INFRACTION)

TECHNICAL SPECIFICATIONS 3.4.3 AND 3.4.4. ALLOW CONTINUED POWER OPERATION FOR 24 HOURS WITH CONTAINMENT SPRAY PUMPS P54B AND P54C (INCLUDING VALVES DIRECTLY ASSOCIATED WITH THE PUMPS WHICH ARE REQUIRED TO FUNCTION DURING ACCIDENT CONDITIONS) SIMULTANEOUSLY INOPERABLE PROVIDED THE EMERGENCY DIESEL GENERATOR (1-2) CONNECTED TO THE OPPOSITE ENGINEERED SAFEGUARDS BUS IS STARTED TO DEMONSTRATE OPERABILITY. IF BOTH PUMPS ARE INOPERABLE AT THE END OF THE 24-HOUR PERIOD, THE REACTOR IS REQUIRED TO BE PLACED IN THE HOT STANDBY CONDITION WITHIN 12 HOURS. CONTRARY TO THE ABOVE, THE PLANT OPERATED AT APPROXIMATELY 88% POWER DURING THE 36-HOUR INTERVAL FROM 7:36 P.M. ON JULY 25, 1980 TO 7:36 A.M. ON JULY 27, 1980, WITH CONTAINMENT SPRAY PUMPS P54B AND P54C INOPERABLE WITH RESPECT TO THEIR DESIGN FUNCTION DUE TO CONTAINMENT SUMP SUCTION VALVE CV 3030 BEING MISPOSITIONED IN THE OPEN POSITION. THE EMERGENCY DIESEL GENERATOR (1-2) CONNECTED TO THE OPPOSITE ENGINEERED SAFEGUARDS BUS WAS NOT STARTED DURING THAT INTERVAL TO DEMONSTRATE OPERABILITY NOR WAS PLANT SHUTDOWN INITIATED.
(8012 INFRACTION)

TECHNICAL SPECIFICATION 3.20.1, PERMITS OPERATION UP TO 72 HOURS WITH A SINGLE SAFETY-RELATED SNUBBER INOPERABLE; REQUIRING THE REACTOR BE IN COLD SHUTDOWN WITHIN 36 HOURS AFTER THESE CONDITIONS CANNOT BE MET. CONTRARY TO THE ABOVE, AFTER TWO SAFETY-RELATED SNUBBERS (NOS. 5 AND 9) WERE DETERMINED INOPERABLE AT 1400 ON MAY 12, 1980, THE LICENSEE NEITHER RESTORED AT LEAST ONE SNUBBER TO OPERABILITY NOR PLACED THE REACTOR IN COLD SHUTDOWN WITHIN 36 HOURS.
(8015 INFRACTION)

TECHNICAL SPECIFICATION 6.9.2 CONTAINS REQUIREMENTS FOR REPORTING SELECTED OCCURRENCES TO NRC. (A) PARAGRAPH 6.9.2.A(2) REQUIRES NOTIFICATION WITHIN 24 HOURS FOR OPERATION WITH ANY PARAMETER SUBJECT TO A LIMITING CONDITION FOR OPERATION (LCO) LESS CONSERVATIVE THAN THE LEAST CONSERVATIVE ASPECT OF THE LCO. CONTRARY TO THE ABOVE, NOTIFICATION WAS NOT MADE WITHIN 24 HOURS AFTER A LEAST CONSERVATIVE ASPECT OF T.S. 3.20.1, PERMITTING OPERATION ABOVE COLD SHUTDOWN WITH A SINGLE SAFETY-RELATED SNUBBER INOPERABLE, WAS EXCEEDED ON MAY 12, 1980 BY OPERATION ABOVE COLD SHUTDOWN WITH TWO INOPERABLE SAFETY-RELATED SNUBBERS; AND (B) PARAGRAPH 6.9.2.B(2) REQUIRES A WRITTEN REPORT WITHIN 30 DAYS FOR CONDITIONS LEADING TO OPERATION IN A DEGRADED MODE PERMITTED BY AN LCO. CONTRARY TO THE ABOVE, NO WRITTEN REPORT WAS SUBMITTED WITHIN 30 DAYS AFTER THE PLANT WAS HEATED ABOVE COLD SHUTDOWN ON MAY 10, 1980, WITH AN INOPERABLE SAFETY-RELATED SNUBBER; A DEGRADED MODE PERMITTED BY T.S. 3.20.1.C.
(8015 DEFICIENCY)

TECHNICAL SPECIFICATION PART 6.8.1(D) STATES, IN PART: "WRITTEN PROCEDURES SHALL BE...IMPLEMENTED...COVERING THE...EMERGENCY PLAN IMPLEMENTATION." CONTRARY TO THE ABOVE, ON AUGUST 28, 1980, THE INSPECTORS VERIFIED BY DIRECT OBSERVATION DURING A TOUR OF SAFETY RELATED AREAS THAT FIRE PLAN IMPLEMENTING PROCEDURES PARTS 8.2 AND 8.3 CONCERNING THE CONTROL OF COMBUSTIBLE MATERIAL WERE NOT BEING IMPLEMENTED WHEN ACCUMULATIONS OF COMBUSTIBLE MATERIAL AND IMPROPERLY STORED COMBUSTIBLE MATERIAL WERE OBSERVED IN SAFETY RELATED AREAS.
(8017 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

OPERATING ROUTINELY.

♦LAST IE SITE INSPECTION DATE: AUGUST 26-29, 1980

♦ INSPECTION REPORT NO: 80-17

SUMMARY

INSPECTION SUMMARY

♦ INSPECTION ON SEPTEMBER 8-12, (80-16): INCLUDED A REVIEW OF THE SECURITY PLAN; SECURITY PROGRAM AUDIT; SECURITY ORGANIZATION; PROTECTED AREA PHYSICAL BARRIERS; VITAL AREA PHYSICAL BARRIERS; LIGHTING; ACCESS CONTROL (IDENTIFICATION, AUTHORIZATION AND BADGING); ACCESS CONTROL (SEARCH); ACCESS CONTROL (ESCORTING); ACCESS CONTROL (VITAL AREAS); ALARM STATIONS; DETECTION AIDS; ASSESSMENT AIDS; COMMUNICATIONS; TESTING AND MAINTENANCE; COMPENSATORY MEASURES; POWER SUPPLY; RESPONSE; LOCKS, KEYS AND COMBINATIONS; RECORDS AND REPORTS; LICENSEE ACTION ON PREVIOUS INSPECTION FINDINGS; AND INDEPENDENT INSPECTION EFFORT. THE INSPECTION INVOLVED 76 INSPECTOR-HOURS ONSITE BY TWO NRC INSPECTORS. THE INSPECTION BEGAN DURING THE BACKSHIFT; 18 INSPECTOR-HOURS WERE ACCOMPLISHED DURING OFF-SHIFT PERIODS. THE LICENSEE WAS FOUND TO BE IN COMPLIANCE WITH NRC REQUIREMENTS WITHIN THE AREAS EXAMINED DURING THE INSPECTION.

♦ INSPECTION ON AUGUST 26-29, (80-17): ROUTINE, UNANNOUNCED INSPECTION TO REVIEW THE IMPLEMENTATION OF THE LICENSEE'S PROGRAM FOR FIRE PROTECTION AND PREVENTION INCLUDING REVIEW OF ADMINISTRATIVE PROCEDURE TO CONTROL COMBUSTIBLE MATERIALS AND IGNITION SOURCES; FIRE BRIGADE TRAINING AND DRILLS; FIRE EMERGENCY PROCEDURES AND EQUIPMENT; GENERAL EMPLOYEE AND CONTRACTOR TRAINING; FIRE INSPECTIONS, QA AUDITS, TECHNICAL SPECIFICATION SURVEILLANCE; AND LICENSEE ACTION ON INSPECTION REPORT NO. 50-255/79-20. THE INSPECTION INVOLVED 52 INSPECTOR-HOURS ONSITE BY TWO NRC INSPECTORS INCLUDING ONE INSPECTOR-HOUR ONSITE DURING OFF-SHIFTS. OF THE SEVEN AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE WERE IDENTIFIED IN SIX AREAS; ONE APPARENT ITEM OF NONCOMPLIANCE WAS IDENTIFIED IN ONE AREA (FAILURE TO IMPLEMENT FIRE PREVENTION PROCEDURES).

NRC POWER RESTRICTIONS: 83% OF THERMAL POWER

REPORTS RECEIVED FROM LICENSEE

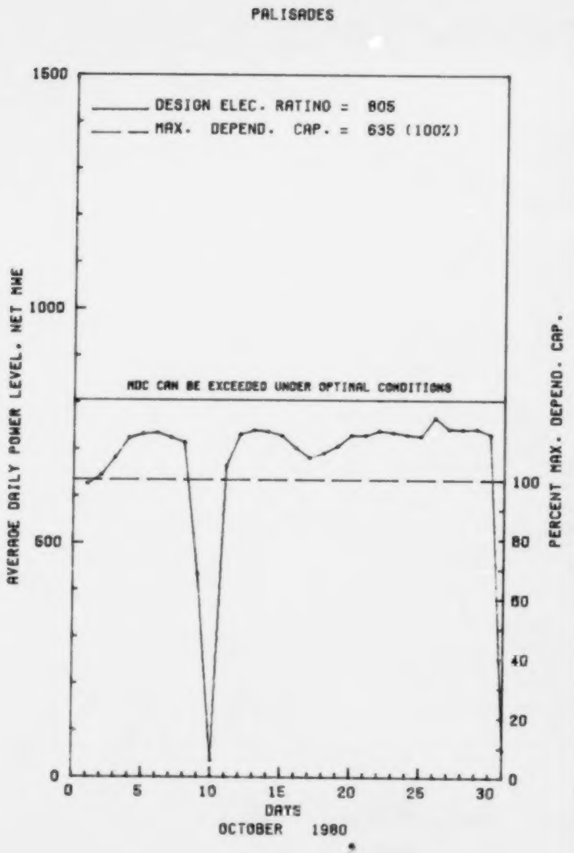
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-36/ 01T-0	09/29/80	10/13/80	A RADIOACTIVE GASEOUS RELEASE OCCURRED WHILE TESTING C-51A VACUUM DEGASIFIER PUMP.
80-37/ 01X-0	10/09/80	10/10/80	PIPE SUPPORT SWAY STRUTS FURN. BY CORNER AND LADA, INC. HAVE THE POTENTIAL OF THE BUSHING BECOMING LOOSE AND/OR DISENGAGED FROM THE CLAMP END OF THE SWAY STRUT.

REVIEWED BY OMPA LOEB: RnR Beebe **DATE:** 12/10/80

REVIEWED BY NRR DL: J.V. Wambach **DATE:** 12/11/80

PALISADES

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-255
2. Utility Contact: P. J. Bodtke (616) 764-8913
3. Licensed Thermal Power (MWt): 2530
4. Nameplate Rating (Gross MWe): 955 × 0.85 = 812
5. Design Electrical Rating (Net MWe): 805
6. Maximum Dependable Capacity (Gross MWe): 675
7. Maximum Dependable Capacity (Net MWe): 635
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	745.0	7,320.0	77,751.0
12. Hours Reactor Was Critical	706.2	3,467.3	43,032.1
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	699.7	3,346.5	40,773.0
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,663,008	7,271,496	78,782,976
17. Gross Electrical Energy (MWH)	524,270	2,247,680	24,279,910
18. Net Electrical Energy (MWH)	495,055	2,111,648	22,782,423
19. Unit Service Factor	93.9	45.7	52.4
20. Unit Availability Factor	93.9	45.7	52.4
21. Unit Capacity Factor (MDC Net)	104.6	45.4	46.1
22. Unit Capacity Factor (DER Net)	82.5	35.8	36.4
23. Unit Forced Outage Rate	3.8	10.9	34.7

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): TMI MODS. - JANUARY 1981.
25. If Shut Down At End of Report Period, Estimated Date of Startup: 11/13/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
19	10/09/80	F	27.3	A	3				SEVERED CABLES IN SWITCHYARD RESULTING IN PLANT TRIP.
20	10/31/80	S	18.0	B	1				TURBINE INSPECTION.

SUMMARY

PALISADES OPERATED NORMALLY DURING OCTOBER WITH TWO OUTAGES ON THE 9TH AND THE 31ST FOR EQUIPMENT FAILURE AND MAINTENANCE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

PEACH BOTTOM 2

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Pennsylvania/York 19 miles S of Lancaster
2.	Electric Reliability Council	Mid-Atlantic Area Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	3293
5.	Design Electrical Rating (MWE Net)	1065
6.	Date of Initial Criticality	September 16, 1973
7.	Date of Commercial Operation	July 5, 1974
8.	Condenser Cooling Method	Once Through with Cooling Towers
9.	Condenser Cooling Water Source	Conowingo Pond/Susquehanna River

Utility & Contractor Information

10.	Licensee	Philadelphia Electric Company
11.	Corporate Address	2301 Market Street Philadelphia, Pennsylvania 19101
12.	Corporate Contact	S. L. Daltroff, Vice President Electric Production
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	GE
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	IE Section Chief	E. McCabe
19.	IE Resident Inspector	C. Cowgill 717-456-7614
20.	Licensing Project Manager	D. Verrelli
21.	Docket Number	50-277
22.	License Number/Date of Issue	DPR-44, 8/8/73
23.	Public Document Room	Government Publications Section State Library of Pennsylvania Education Building Commonwealth and Walnut Street Harrisburg, Pennsylvania 17126

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO THE LICENSEE'S QUALITY ASSURANCE PLAN, VOL. III, SECTION 2, PARAGRAPH 6.1, PROCEDURES HPO/CO 9, HPO/CO 9A, AND HPO/CO 9B WERE NOT IN USE AT THE RESPIRATOR QA STATION.
(8011 INFRACTION)

CONTRARY TO TECHNICAL SPECIFICATION 6.11, RADIATION PROTECTION PROCEDURES WERE NOT FOLLOWED IN THAT A NEGATIVE PRESSURE TEST WAS NOT DONE PRIOR TO USE OF THE RESPIRATOR AS REQUIRED BY HPO/CO 9A.
(8011 INFRACTION)

10 CFR 19.11(A) AND (B) REQUIRE THAT CURRENT COPIES OF PART 19, PART 20, LICENSE CONDITIONS, DOCUMENTS INCORPORATED INTO THE LICENSE, LICENSE AMENDMENTS AND OPERATING PROCEDURES BE POSTED, OR THAT A NOTICE DESCRIBING THESE DOCUMENTS AND WHERE THEY MAY BE EXAMINED, BE POSTED. 10 CFR 19.11(C) REQUIRES THAT A FORM NRC-3, "NOTICE TO EMPLOYEES" BE POSTED. CONTRARY TO THE ABOVE REQUIREMENTS A CHECK OF THE SINGLE CONTROLLED BULLETIN BOARD DURING SITE ENTRY ON APRIL 22, 1980, FOUND 10 CFR 19 NOT CURRENT, AND A FORM AEC-3 INSTEAD OF FORM NRC-3 POSTED.
(8011 DEFICIENCY)

TECHNICAL SPECIFICATION 6.8.1 REQUIRES IN PART: "WRITTEN PROCEDURES AND ADMINISTRATIVE POLICIES SHALL BE ESTABLISHED, IMPLEMENTED, AND MAINTAINED...." PROCEDURE A-30, "PLANT HOUSEKEEPING CONTROLS," REVISION 1, DATED JANUARY 14, 1980, STATES IN PART: "HOUSEKEEPING REQUIREMENTS: 8...THE STORAGE OF EQUIPMENT AND MATERIAL SHALL NOT IMPEDE ACCESSIBILITY OF FIRE FIGHTING EQUIPMENT...." CONTRARY TO THE ABOVE, THE INSPECTOR IDENTIFIED THE FOLLOWING CONDITIONS: ON MAY 7, 1980 FIRE STATION 135-30, LOCATED IN THE RECIRCULATION PUMP M.G. SET ROOM, WAS OBSTRUCTED BY A LARGE MOBILE TOOL CABINET. ON MAY 15, 1980 FIRE STATION 165-32, LOCATED ON THE 165 FOOT ELEVATION IN THE REACTOR BUILDING WAS OBSTRUCTED BY SCAFFOLDING MATERIAL, A LARGE CABLE REEL AND A LADDER.
(8014 INFRACTION)

TECHNICAL SPECIFICATION 6.9.2 "REPORTABLE OCCURRENCES," REQUIRES IN PART, PROMPT NOTIFICATION WITH WRITTEN FOLLOWUP IN EVENT OF PERFORMANCE ON COMPONENTS REQUIRING REMEDIAL ACTION TO PREVENT OPERATION IN A MANNER LESS CONSERVATIVE THAN ASSUMED IN THE SAFETY ANALYSIS REPORT. CONTRARY TO THE ABOVE, ON NINE OCCASIONS THE LICENSEE WAS INFORMED BY THE ARCHITECT-ENGINEER OF SUCH A CONDITION; IN THAT SEISMICALLY-DESIGNED PIPE SUPPORTS, AS BUILT, WOULD FAIL UNDER DESIGN BASIS EARTHQUAKE CONDITIONS; AND DID NOT MAKE PROMPT REPORT WITH WRITTEN FOLLOWUP.
(8024 DEFICIENCY)

FAILURE TO MAINTAIN AN ISOLATION ZONE ON JULY 29, 1980.
(8024 INFRACTION)

FAILURE TO PROPERLY POST A RADIOLOGICAL RESTRICTED AREA.
(8026 DEFICIENCY)

FAILURE TO POSSESS PROPER DOSIMETRIC DEVICES.
(8026 DEFICIENCY)

FAILURE TO LOG REASON FOR ENTRY INTO NORMALLY UNOCCUPIED VITAL AREAS.
(8030 DEFICIENCY)

FAILURE TO LIMIT UNESCORTED ACCESS TO VITAL AREAS TO THOSE PERSONNEL WITH A PICTURE BADGE.
(8030 INFRACTION)

FAILURE TO LIMIT USE OF NON PICTURE BADGES FOR UNESCORTED ACCESS TO SEVEN DAYS.
(8030 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS:

+ THE REACTOR OPERATED AT ABOUT 100 PERCENT POWER WITH A UNIT OFFGAS RATE OF 2970 MICROCURIES PER SECOND AND A COMBINED UNIT 2 AND UNIT 3 STACK GAS RELEASE RATE OF 230 MICROCURIES PER SECOND.

+ LAST IE SITE INSPECTION DATE: 10/1-31/80

+ INSPECTION REPORT NO: 50-277/80-32

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-013/ 03L	08/18/80	08/29/80	DURING UNIT 2 REFUEL OUTAGE SURVEILLANCE LEAK TEST RESULTS EXCEEDED TECH SPEC LIMITS - SEVERAL LEAKING VALVES WERE IDENTIFIED, REPAIRED AND RETESTED SATISFACTORILY.
80-014/ 04T	08/08/80	08/18/80	OIL SHEEN NOTED ON CONOWINGO PUMP - SOURCE (DIESEL TANK OVERFLOW) FOUND AND CORRECTED. ACCESSIBLE OIL REMOVED. EPA NOTIFIED.

SUMMARY

INSPECTION SUMMARY

+ 50-277/80-08 - APR 1-30: ROUTINE, ONSITE REGULAR AND BACKSHIFT INSPECTION BY THE RESIDENT INSPECTORS (64 HRS). AREAS INSPECTED INCLUDED PLANT OPERATIONS, FACILITY TOURS, CONTROL ROOM INSPECTIONS, REVIEW OF PERIODIC REPORTS, HOUSEKEEPING, RADIATION PROTECTION, REACTOR CHEMISTRY, PHYSICAL SECURITY, LER REVIEW ONSITE AND FOLLOWUP ON PRIOR IDENTIFIED ITEMS. TWO NONCOMPLIANCES WERE IDENTIFIED: FAILURE TO FOLLOW RWP REQUIREMENTS; FAILURE TO FOLLOW PROCEDURES RELATIVE TO SMOKING IN PLANT.

+ 50-277/80-24 - JUL 1-31: ROUTINE, ONSITE REGULAR AND BACKSHIFT INSPECTIONS BY THE RESIDENT INSPECTORS AND REGION I MANAGEMENT (64 HRS). AREAS INSPECTED INCLUDED: FOLLOWUP ON PRIOR IDENTIFIED ITEMS, OPERATIONAL SAFETY VERIFICATION, FACILITY TOURS, REACTOR CHEMISTRY, IE BULLETIN AND CIRCULAR FOLLOWUP, RADIATION PROTECTION, PHYSICAL SECURITY, TRAINING, STAFF LICENSE APPLICATIONS, IMMEDIATE ACTION LETTER FOLLOWUP, REVIEW OF PERIODIC AND SPECIAL REPORTS, AND NRC MEETINGS WITH CORPORATE STAFF. TWO NONCOMPLIANCES WERE IDENTIFIED: FAILURE TO MAINTAIN ISOLATION ZONE CLEAR; FAILURE TO REPORT SEISMIC RESTRAINT INADEQUACIES IN ACCORDANCE WITH TECHNICAL SPECIFICATIONS AND IEB 79-14.

+ 50-277/80-28 - AUG 1-31: ROUTINE, ONSITE REGULAR AND BACKSHIFT INSPECTIONS BY THE RESIDENT INSPECTORS (53 HRS). AREAS INSPECTED INCLUDED ACCESSIBLE PORTIONS OF THE FACILITY, RADIATION PROTECTION, PHYSICAL SECURITY, OPERATIONAL SAFETY VERIFICATION, FACILITY TOURS, HOUSEKEEPING, IE BULLETIN FOLLOWUP, CONTROL ROOM OBSERVATIONS, LER REVIEW ONSITE AND IN-OFFICE, FOLLOWUP ON PREVIOUSLY IDENTIFIED ITEMS, AND REVIEW OF PERIODIC AND SPECIAL REPORTS. THREE NONCOMPLIANCES WERE IDENTIFIED: FAILURE TO FOLLOW HOUSEKEEPING PROCEDURES; FAILURE TO POSITIVELY CONTROL A DESIGNATED VEHICLE; LATE SUBMITTAL OF THERMAL MAPPING REPORTS.

+ 50-277/80-29 - SEP 1-30: ROUTINE, ONSITE REGULAR AND BACKSHIFT INSPECTIONS BY THE RESIDENT INSPECTORS (56 HRS). AREAS INSPECTED INCLUDED ACCESSIBLE PORTIONS OF THE FACILITY, RADIATION PROTECTION, PHYSICAL SECURITY, OPERATIONAL SAFETY VERIFICATION, FACILITY TOURS, CONTROL ROOM OBSERVATIONS, LER REVIEW ONSITE AND IN-OFFICE, IE BULLETIN FOLLOWUP, AND REVIEW OF PERIODIC AND SPECIAL REPORTS. TWO NONCOMPLIANCES WERE IDENTIFIED: FAILURE TO MAINTAIN SECONDARY CONTAINMENT; FAILURE TO FOLLOW ESCORT PROCEDURES.

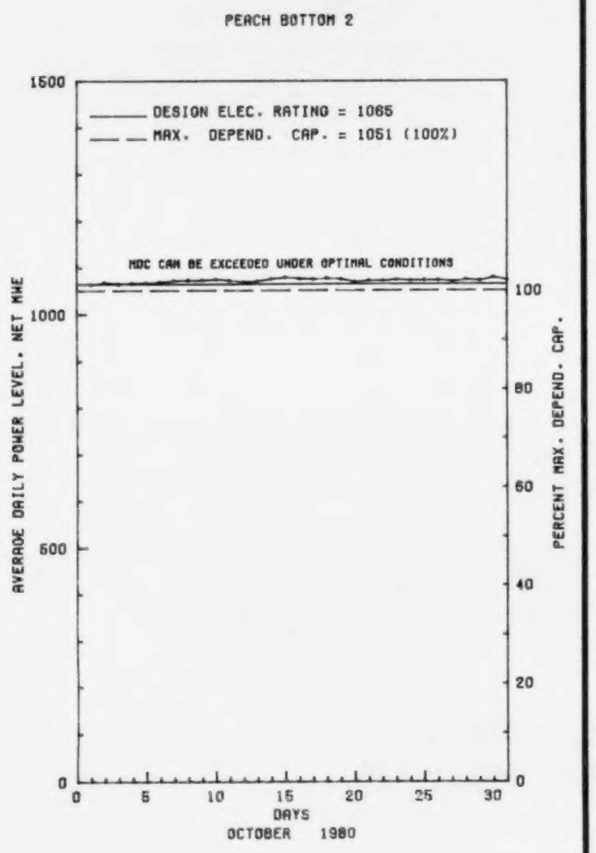
+ 50-277/80-30 - SEP 22-26: ROUTINE, UNANNOUNCED, PHYSICAL SECURITY INSPECTION BY ONE REGION-BASED INSPECTOR (29 HRS) INCLUDING REVIEW OF THE SECURITY PLAN; SECURITY PROGRAM AUDIT; SECURITY ORGANIZATION; PHYSICAL BARRIERS, (PROTECTED AREA) AND (VITAL AREAS); LIGHTING; ACCESS CONTROL, (SEARCH) (ESCORTING) (IDENTIFICATION, AUTHORIZATION AND BADGING) AND (VITAL AREAS); ALARM STATIONS; DETECTION AIDS; COMMUNICATIONS; TESTING AND MAINTENANCE; COMPENSATORY MEASURES; RESPONSE, LOCKS, KEYS AND COMBINATIONS; AND RECORDS AND REPORTS. THREE NONCOMPLIANCES WERE IDENTIFIED: FAILURE TO LIMIT USE OF BADGES AS REQUIRED; FAILURE TO OBSERVE BADGE ISSUANCE PROCEDURE; FAILURE TO MAINTAIN REQUIRED ACCESS CONTROL LOG.

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 12-10-80

REVIEWED BY NRR DL: P. Erickson **DATE:** 12-11-80

PEACH BOTTOM 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-277
2. Utility Contact: W. M. Alden (215) 841-5022
3. Licensed Thermal Power (MWt): 3293
4. Nameplate Rating (Gross MWe): 1280 x 0.9 = 1152
5. Design Electrical Rating (Net MWe): 1065
6. Maximum Dependable Capacity (Gross MWe): 1098
7. Maximum Dependable Capacity (Net MWe): 1051
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	745.0	7,320.0	55,464.0
12. Hours Reactor Was Critical	745.0	3,424.0	41,179.0
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	745.0	3,206.3	40,189.1
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	2,445,286	9,521,479	116,287,123
17. Gross Electrical Energy (MWH)	828,800	3,165,820	38,224,050
18. Net Electrical Energy (MWH)	798,720	3,008,404	36,624,189
19. Unit Service Factor	100.0	43.8	72.5
20. Unit Availability Factor	100.0	43.8	72.5
21. Unit Capacity Factor (MDC Net)	102.0	39.1	62.8
22. Unit Capacity Factor (DER Net)	100.7	38.6	62.0
23. Unit Forced Outage Rate	.0	1.7	6.0

24. Shutdowns Scheduled Over Next 6 Months (Type,Date,and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	NONE								

SUMMARY

PEACH BOTTOM 2 OPERATED AT OR ABOVE FULL POWER FOR OCTOBER WITH NO REPORTABLE REDUCTIONS OR OUTAGES.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

PEACH BOTTOM 3

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Pennsylvania/York 19 miles S of Lancaster
2.	Electric Reliability Council	Mid-Atlantic Area Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	3293
5.	Design Electrical Rating (MWE Net)	1065
6.	Date of Initial Criticality	August 7, 1974
7.	Date of Commercial Operation	December 23, 1974
8.	Condenser Cooling Method	Once Through with Cooling Towers
9.	Condenser Cooling Water Source	Conowingo Pond/Susquehanna River

Utility & Contractor Information

10.	Licensee	Philadelphia Electric Company
11.	Corporate Address	2301 Market Street Philadelphia, Pennsylvania 19101
12.	Corporate Contact	S. L. Daltroff, Vice President Electric Production
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	GE
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	IE Section Chief	E. McCabe
19.	IE Resident Inspector	C. Cowgill (717-456-7614)
20.	Licensing Project Manager	D. Verrelli
21.	Docket Number	50-278
22.	License Number/Date of Issue	DPR-56, 7/2/74
23.	Public Document Room	Government Publications Section State Library of Pennsylvania Education Building Commonwealth and Walnut Street Harrisburg, Pennsylvania 17126

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO THE LICENSEE'S QUALITY ASSURANCE PLAN, VOL. III, SECTION 2, PARAGRAPH 6.1, PROCEDURES HPO/CO 9, HPO/CO 9A, AND HPO/CO 9B WERE NOT IN USE AT THE RESPIRATOR QA STATION.
(8011 INFRACTION)

CONTRARY TO 10 CFR 19.11(C) A CURRENT FORM NRC-3 WAS NOT POSTED.
(8011 DEFICIENCY)

TECHNICAL SPECIFICATION 3.11.D PERMITS CONTINUED OPERATION FOR ONLY 72 HOURS AFTER DISCOVERY THAT CERTAIN SAFETY-RELATED SNUBBERS ARE INOPERABLE AND LISTS SNUBBER NUMBER 10-GB-5-44 AS AN APPLICABLE SNUBBER.CONTRARY TO THE ABOVE, THE LICENSEE WAS INFORMED BY THE ARCHITECT ENGINEER ON JUNE 5, 1980 THAT SNUBBER 10-GB-5-44 WAS INOPERABLE--PLANT OPERATION CONTINUED FOR 43 DAYS AND THE SNUBBER WAS NOT MADE OPERABLE NOR WAS A PLANT SHUTDOWN PERFORMED.
(8018 INFRACTION)

FAILURE TO MAINTAIN AN ISOLATION ZONE ON JULY 29, 1980.
(8018 INFRACTION)

FAILURE TO LOG REASON FOR ENTRY INTO NORMALLY UNOCCUPIED VITAL AREAS.
(8022 DEFICIENCY)

FAILURE TO LIMIT USE OF NONPICTURE BADGES FOR UNESCORTED ACCESS TO SEVEN DAYS.
(8022 INFRACTION)

FAILURE TO LIMIT UNESCORTED ACCESS TO VITAL AREAS TO THOSE PERSONNEL WITH A PICTURE BADGE.
(8022 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

- SYSTEM AND COMPONENT PROBLEMS:
- THE 3A MAIN TRANSFORMER SHOWED SIGNS OF INSULATION BREAKDOWN AND WAS REPLACED DURING A TEN-DAY OUTAGE.

PLANT STATUS:

- THE REACTOR OPERATED AT ABOUT FULL POWER. ON OCTOBER 19 A TEN-DAY OUTAGE BEGAN FOR REPLACEMENT OF THE 3A MAIN TRANSFORMER. A SCRAM OCCURRED DURING POWER ASCENSION ON OCTOBER 30 DUE TO A TURBINE CONTROL VALVE PROBLEM. UNIT OFFGAS RATE WAS 3630 MICROCURIES PER SECOND AND COMBINED UNIT 2 AND UNIT 3 STACK GAS RELEASE RATE WAS 230 MICROCURIES PER SECOND.

- LAST IE SITE INSPECTION DATE: 10/1-31/80
- INSPECTION REPORT NO: 50-278/80-24

SUMMARY

INSPECTION SUMMARY

- 50-278/80-07 - APR 1-30: ROUTINE, ONSITE REGULAR AND BACKSHIFT INSPECTION BY THE RESIDENT INSPECTORS (57 HRS). AREAS INSPECTED INCLUDED PLANT OPERATIONS, FACILITY TOURS, THE CONTROL ROOM, PERIODIC REPORTS, HOUSEKEEPING, RADIATION PROTECTION, REACTOR CHEMISTRY, PHYSICAL SECURITY, LER REVIEW ONSITE AND PRIOR ITEMS. ONE NONCOMPLIANCE WAS IDENTIFIED: FAILURE TO MAINTAIN THE FIRE RETARDANT SEAL ON A ELECTRICAL CABLE PENETRATION.
- 50-278/80-18 - JUL 1-31: ROUTINE, ONSITE REGULAR AND BACKSHIFT INSPECTIONS BY THE RESIDENT INSPECTORS AND REGION I MANAGEMENT (63 HRS). AREAS INSPECTED INCLUDED: PRIOR ITEMS, OPERATIONAL SAFETY VERIFICATION, FACILITY TOURS, REACTOR CHEMISTRY, IE BULLETIN AND CIRCULAR FOLLOWUP, RADIATION PROTECTION, PHYSICAL SECURITY, TRAINING, STAFF LICENSE APPLICATIONS, IMMEDIATE ACTION LETTER FOLLOWUP, REVIEW OF PERIODIC AND SPECIAL REPORTS, AND NRC MEETINGS WITH CORPORATE STAFF. TWO NONCOMPLIANCES WERE IDENTIFIED: FAILURE TO INITIATE A SHUTDOWN WITH AN INOPERABLE SNUBBER IN ACCORDANCE WITH TECHNICAL SPECIFICATION LCO; FAILURE TO MAINTAIN ISOLATION ZONE CLEAR.
- 50-278/80-20 - AUG 1-31: ROUTINE, ONSITE REGULAR AND BACKSHIFT INSPECTIONS BY THE RESIDENT INSPECTORS (53 HRS). AREAS INSPECTED INCLUDED ACCESSIBLE PORTIONS OF THE FACILITY, RADIATION PROTECTION, PHYSICAL SECURITY, OPERATIONAL SAFETY, FACILITY TOURS, HOUSEKEEPING, IE BULLETIN FOLLOWUP, CONTROL ROOM OBSERVATIONS, LER REVIEW ONSITE AND IN-OFFICE, PREVIOUSLY IDENTIFIED ITEMS, PERIODIC AND SPECIAL REPORTS. THREE NONCOMPLIANCES WERE IDENTIFIED: FAILURE TO POSITIVELY CONTROL A DESIGNATED VEHICLE; LATE SUBMITTAL OF LER'S; LATE SUBMITTAL OF THERMAL MAPPING REPORTS.
- 50-278/80-21 - SEP 1-30: ROUTINE, ONSITE REGULAR AND BACKSHIFT INSPECTION BY THE RESIDENT INSPECTORS (56 HRS). AREAS INSPECTED INCLUDED ACCESSIBLE PORTIONS OF THE FACILITY, RADIATION PROTECTION, PHYSICAL SECURITY, OPERATIONAL SAFETY VERIFICATION, FACILITY TOURS, CONTROL ROOM OBSERVATIONS, LER REVIEW ONSITE AND IN-OFFICE. IE BULLETIN FOLLOWUP, AND REVIEW OF PERIODIC AND SPECIAL REPORTS. ONE NONCOMPLIANCE WAS IDENTIFIED: FAILURE TO FOLLOW ESCORT PROCEDURES.
- 50-278/80-22 - SEP 22-26: ROUTINE, UNANNOUNCED PHYSICAL SECURITY INSPECTION BY ONE REGION-BASED INSPECTOR (29 HRS) INCLUDING THE SECURITY PLAN; SECURITY PROGRAM AUDIT; SECURITY ORGANIZATION; PHYSICAL BARRIERS, (PROTECTED AREA) AND (VITAL AREAS); LIGHTING; ACCESS CONTROL, (SEARCH) (ESCORTING) (IDENTIFICATION, AUTHORIZATION AND BADGING) AND (VITAL AREAS); ALARM STATIONS; DETECTION AIDS; COMMUNICATIONS; TESTING AND MAINTENANCE; COMPENSATORY MEASURES; RESPONSE; LOCKS, KEYS AND COMBINATIONS; AND RECORDS AND REPORTS. THREE NONCOMPLIANCES WERE IDENTIFIED: FAILURE TO LIMIT USE OF BADGES AS REQUIRED; FAILURE TO OBSERVE BADGE ISSUANCE PROCEDURE; FAILURE TO MAINTAIN REQUIRED ACCESS CONTROL LOG.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

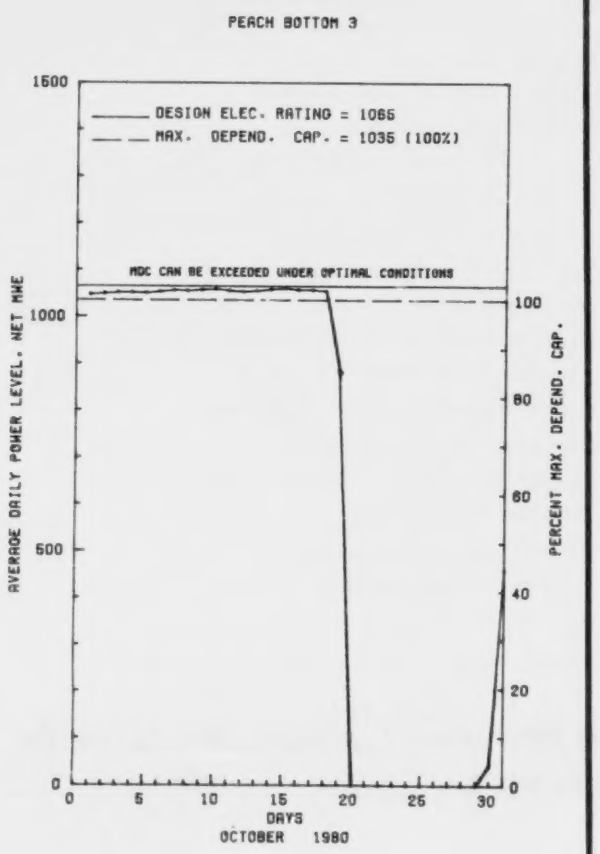
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-011/ 03L	05/16/80	06/13/80	PS3-2-128B INPUT TO ISOLATE SHUTDOWN COOLING ON HIGH REACTOR PRESSURE, SETPOINT HAD DRIFTED HIGH - INSTRUMENT RECALIBRATED AND TESTED.
80-012/ 03L	05/20/80	06/19/80	PRESSURE SWITCH PS5121B, PROVIDING RPS INPUT FOR TURBINE CONTROL VALVE FAST CLOSURE, FOUND SET TOO HIGH - SETPOINT IMMEDIATELY ADJUSTED TO WITHIN ALLOWABLE RANGE.
80-015/ 03L	07/07/80	08/06/80	3A REFUEL FLOOR EXHAUST RAD MONITOR FOUND DRIFTING DOWNSCALE - MONITOR PLACED IN TRIPPED (SAFE) CONDITION, REPAIRED, CALIBRATED, AND TESTED SATISFACTORILY.
80-017/ 03L	08/02/80	09/01/80	2 INCH DRYWELL VENT VALVE FAILED TO CLOSE. CAUSE WAS DUE TO TIGHT PACKING. PACKING LOOSENED AND VALVE OPERATED NORMALLY. VALVE WAS SUCCESSFULLY LEAK TESTED.
80-018/ 03L	08/02/80	09/01/80	REACTOR WATER CLEANUP SYSTEM RETURN ISOLATION VALVE FAILED TO CLOSE FROM THE CONTROL SWITCH. VALVE WAS CLOSED AND ELECTRICALLY DISARMED.
80-019/ 03L	08/06/80	09/05/80	RHR INJECTION VALVE FAILED TO OPEN. REDUNDANT EQUIPMENT WAS TESTED. PLANT SHUTDOWN ON 8/15 FOR REPAIRS. THE VALVE LIMITORQUE CAUSED THE FAILURE. LIMITORQUE REPAIRED.
80-020/ 03L	08/15/80	09/12/80	CONTAINMENT VENTILATION VALVE FAILED A LEAK TEST. THE CAUSE OF FAILURE WAS CORRECTED AND THE VALVE WAS SUCCESSFULLY STROKED AND LEAK TESTED.
80-021/ 03L	08/18/80	09/17/80	CONTAINMENT VENTILATION VALVE FAILED TO CLOSE. CAUSE OF THE FAILURE WAS A SOLENOID VALVE WHICH HAD STUCK IN ENERGIZED POSITION. SOLENOID WAS REPAIRED AND VENT VALVE CYCLED SUCCESSFULLY.

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 12-10-80

REVIEWED BY NRR DL: M. B. Fairtile **DATE:** 12/11/80

PEACH BOTTOM 3

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-278
2. Utility Contact: W. M. Alden (215) 841-5022
3. Licensed Thermal Power (MWt): 3293
4. Nameplat Rating (Gross MWe): 1280 x 0.9 = 1152
5. Design Electrical Rating (Net MWe): 1065
6. Maximum Dependable Capacity (Gross MWe): 1098
7. Maximum Dependable Capacity (Net MWe): 1035
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	745.0	7,320.0	51,360.0
12. Hours Reactor Was Critical	519.0	5,947.0	40,793.0
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	483.6	5,741.8	39,673.6
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,545,144	18,271,436	112,490,590
17. Gross Electrical Energy (MWH)	507,250	6,065,510	36,651,370
18. Net Electrical Energy (MWH)	485,122	5,838,981	35,183,781
19. Unit Service Factor	64.9	78.4	77.2
20. Unit Availability Factor	64.9	78.4	77.2
21. Unit Capacity Factor (MDC Net)	62.9	77.1	66.2
22. Unit Capacity Factor (DER Net)	61.1	74.9	64.3
23. Unit Forced Outage Rate	2.8	11.7	7.6

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
REFUELING/MAINTENANCE, 3/7/81, SIXTEEN WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: 11/01/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type¹	Duration (Hours)	Reason²	Method of Shutting Down Reactor³	Licensee Event Report No.	System Code⁴	Component Code⁵	Cause & Corrective Action to Prevent Recurrence
16	10/19/80	S	247.3	B	1		EB	TRANSF	UNIT WAS SHUTDOWN TO REPLACE '3A' TRANSFORMER.
17	10/30/80	F	13.6	G	3		HA	INSTRU	AUTOMATIC SCRAM DUE TO GENERATOR POWER LOAD UNBALANCE PROJECTION.
18	10/31/80	F	.5	B	9		HA	INSTRU	CORRECT FAILURE TO COMPLETELY REMOVE BLOCKING OF ELECTRIC POWER INSTRUMENT (C.T.) ASSOCIATED WITH OUTAGE NO. 16. REACTOR REMAINED CRITICAL.

SUMMARY

PEACH BOTTOM 3 OPERATED WITH THREE OUTAGES DURING OCTOBER FOR MAINTENANCE AND OPERATIONAL ERROR.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

PILGRIM

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Massachusetts/Plymouth 4 miles SE of Plymouth
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	1998
5.	Design Electrical Rating (MWE Net)	655
6.	Date of Initial Criticality	June 16, 1972
7.	Date of Commercial Operation	December 1972
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Cape Cod Bay

Utility & Contractor Information

10.	Licensee	Boston Edison Company
11.	Corporate Address	800 Boylston Street Boston, Massachusetts 02199
12.	Corporate Contact	A. Corey, Vice President
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	GE
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	IE Section Chief	T. MARTIN
19.	IE Resident Inspector	J. JOHNSON
20.	Licensing Project Manager	M. WILLIAMS x29798
21.	Docket Number	50-293
22.	License Number/Date of Issue	DPR-35, 6/8/72
23.	Public Document Room	Plymouth Public Library North Street Plymouth, Massachusetts 02360

INSPECTION STATUS

ENFORCEMENT STATUS

FAILURE TO FOLLOW SURVEILLANCE AND SHIFT TURNOVER PROCEDURES - VALVE MO-1301-25 OUT OF NORMAL POSITION.
(8025 INFRACTION)

CONTRARY TO TECH SPEC 2.1.A THE APRM FLUX SCRAM SETTINGS WERE SET AS HIGH AS 122.9 PERCENT OF RATED THERMAL POWER.
(8026 INFRACTION)

CONTRARY TO TECH SPEC 6.9.B.2.A A 30 DAY REPORT WAS NOT MADE CONCERNING CHANNEL 'F' APRM 120% CLAMP CALIBRATION DATA DATED 7/1/80.
(8026 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

- THE MAIN STEAM LINE HIGH RADIATION SCRAM OF 10/1/80 WAS ATTRIBUTED TO AIR INTRODUCED INTO THE FEEDWATER SYSTEM UPON PLACING A CONDENSATE DEMINERALIZER IN SERVICE. RECIRCULATION PUMPS FAILED TO RUN BACK TO MINIMUM SPEED DUE TO A WIRING ERROR IN THE FEEDWATER FLOW INDICATION CIRCUIT. 'D' SAFETY-RELIEF VALVE JAMMED OPEN AND THE CAUSE ATTRIBUTED TO FOREIGN MATERIAL IN THE MAIN VALVE BODY INTERNALS.
- ON 10/07/80 THE 'A' SAFETY-RELIEF VALVE (SRV) INADVERTENTLY OPENED DUE TO HIGH NITROGEN SUPPLY PRESSURE. A SIMILAR EVENT WAS EXPERIENCED ON 10/31/80. THE NITROGEN SUPPLY TO SRV'S AND OTHER PROCESS CONTROLS IN THE PRIMARY CONTAINMENT WAS ISOLATED PENDING INVESTIGATION OF THE NITROGEN SUPPLY PROBLEMS (PROPOSED I&E BULLETIN).
- ON 10/30/80 THE ONSITE REVIEW COMMITTEE REVIEWED DESIGN DEFICIENCY OF HAVING A SEISMIC ANCHOR FOR PART OF THE SALT SCREEN WATER SYSTEM OUTSIDE THE MISSILE BARRIER. MODIFICATIONS ARE IN PROGRESS TO INSTALL A SEISMIC ANCHOR INSIDE THE MISSILE BARRIER.

PLANT STATUS:

- AS OF 10/31/80 THE PLANT IS IN HOT SHUTDOWN AWAITING A TECH SPEC CHANGE PROVIDING TEMPORARY RELIEF FROM A SALT SCREEN WATER SYSTEM REQUIREMENT PERTAINING TO TORNADO MISSILE PROTECTION.
- MAIN STACK AND OFF GAS RELEASE RATES DURING FULL POWER OPERATION THIS MONTH WERE APPROXIMATELY 2200 AND 40,000 MICROCURIES/SEC. RESPECTIVELY.

- LAST IE SITE INSPECTION DATE: 10/1-31/80

INSPECTION REPORT NO: 50-293/80-27

SUMMARY

INSPECTION SUMMARY

- 50-293/80-25 - JUL 1-31: ROUTINE, ONSITE, UNANNOUNCED INSPECTION BY TWO RESIDENT INSPECTORS (148) HRS) OF PLANT OPERATIONS INCLUDING AN OPERATIONAL SAFETY VERIFICATION, SURVEILLANCE AND MAINTENANCE OBSERVATION, A REVIEW OF PERIODIC REPORTS, AN IN-OFFICE REVIEW OF LICENSEE EVENT REPORTS, A REVIEW OF LICENSE APPLICATIONS FOR TRAINING STAFF, A REVIEW OF LICENSEE ACTIONS IN RESPONSE TO IE BULLETINS, AND A REVIEW OF TEMPERATURE EFFECTS ON REACTOR VESSEL INSTRUMENTATION. ONE NONCOMPLIANCE WAS IDENTIFIED: FAILURE TO FOLLOW SURVEILLANCE AND SHIFT TURNOVER PROCEDURES WHICH SPECIFY VALVE POSITIONS.
- 50-293/80-26 - AUG 1-31: ROUTINE, ONSITE, UNANNOUNCED INSPECTION BY TWO RESIDENT INSPECTORS (96 HRS) OF PLANT OPERATIONS INCLUDING AN OPERATIONAL SAFETY VERIFICATION, FOLLOWUP ON SURVEILLANCE AND MAINTENANCE ACTIVITIES, AN IN-OFFICE REVIEW AND FOLLOWUP OF LICENSEE EVENT REPORTS, ACTIONS IN RESPONSE TO IE BULLETINS, A REVIEW OF PERSONNEL AND ORGANIZATION CHANGES, AND A REVIEW OF PROCESS COMPUTER INPUT DATA INVOLVING FUEL LENGTHS. THREE NONCOMPLIANCES WERE IDENTIFIED: FAILURE TO SET APRM FLUX SCRAM SETTINGS IN ACCORDANCE WITH TECHNICAL SPECIFICATIONS; FAILURE TO REPORT APRM CHANNEL 'F' 120% CLAMP TRIP SETPOINT OUTSIDE TECHNICAL SPECIFICATION LIMITS; FAILURE TO PROPERLY ESCORT VISITORS.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

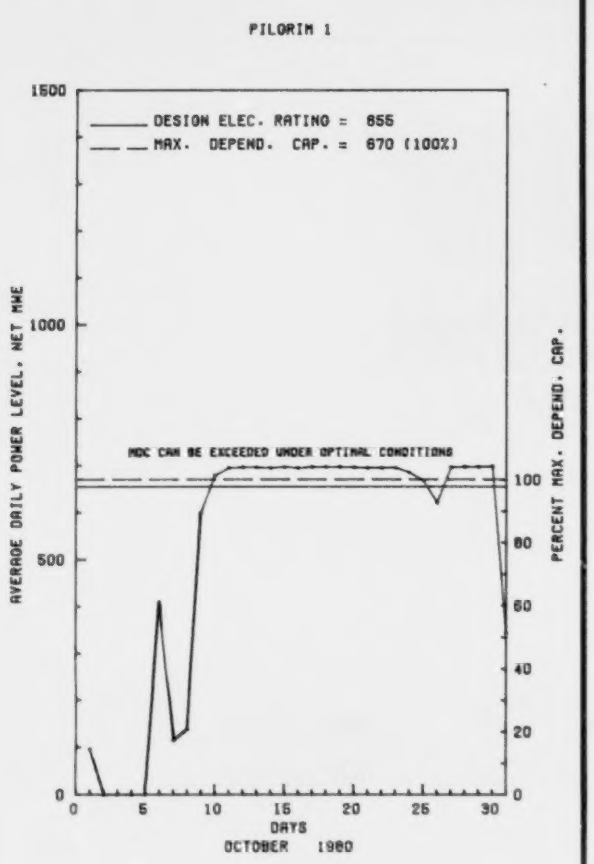
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-032/ 01T	08/24/80	09/22/80	DRYWELL AMBIENT AND YARWAY REFERENCE LEG TEMPERATURES HIGHER THAN DESIGN CAUSING ACTUAL LEVEL TO BE LOWER THAN INDICATED
80-050/ 01T	09/11/80	09/19/80	LOSS OF ALTERNATE SHUTDOWN ABILITY FOR RBCCW AND SSW DUE TO MAINTENANCE ON REDUNDANT EQUIPMENT
80-054/ 03L	09/01/80	09/16/80	REVISED T.S. ORGANIZATION DELETING NUC. OPS. MGR. FROM ORC AND PUTTING HIM IN NSRAC
80-055/ 03L	08/21/80	09/19/80	RCIC MOV OVERLOAD ANNUNCIATOR SENSOR (THERMAL) FAILED
80-056/ 03L	08/21/80	09/19/80	VOLTAGE CONTROL CARD FAILURE A 125VDC BATT CHARGER INOPERABLE
80-057/ 03L	05/21/80	09/19/80	'A' RECIRC PUMP TRIP. BURNED OUT DC COIL IN RELAY
80-058/ 03L	08/21/80	09/19/80	HPCI CONTROL OSCILLATING
80-059/ 03L	09/12/80	09/23/80	FAILURE TO SAMPLE SBGT CHARCOAL FILTERS
80-060/ 03L	08/25/80	09/23/80	HPCI CONTROL OSCILLATING. DIRT IN CONTROL OIL SYSTEM
80-061/ 03L	08/25/80	09/24/80	CORE SPRAY MOV24A FAILED DURING COMPENSATORY SURVEILLANCE. VALVE LOCKED IN OPEN POSITION (NORMAL) AND SYSTEM INOPERABLE DURING REPAIR OF VALVE
80-062/ 03L	09/03/80	10/03/80	'A' EDG INOPERABLE DUE TO BROKEN FUEL LINE
80-063/ 03L	08/28/80	09/29/80	CONTAINMENT ATMOSPHERE SAMPLER C-19 INOP DUE TO MOISTURE FROM STEAM PACKING LEAK IN DRYWELL
80-064/ 03L	08/29/80	09/29/80	REACTOR BLDG AIRLOCK DOOR INTERLOCK INOPERABLE
80-065/ 03L	08/28/80	09/29/80	NITROGEN PURGE VALVE A05033B WOULD NOT CLOSE DUE TO DEFECTIVE SOLENOID VALVE
80-066/ 03L	09/01/80	09/30/80	DIESEL FIRE PUMP INOPERABLE DUE TO GOVERNOR LINKAGE BROKEN
80-067/ 01T	10/02/80	10/10/80	SEISMIC ANALYSIS B RHR SYSTEM REVEALED THAT DUE TO ERROR IN ANALYSIS THE B RHR SYSTEM WOULD NOT FUNCTION DURING SSE
80-068/ 03L	09/24/80	09/30/80	FAILURE TO LOG RPS MG SET VOLTS/AMPS ONCE PER SHIFT
80-069/ 01T	10/07/80	10/20/80	'A' SRV OPENED AT POWER DUE TO HIGH NITROGEN SUPPLY PRESSURE
80-070/ 01T	10/10/80	10/22/80	CIRCUIT BREAKER B204 TRIPPED TWICE DEENERGIZING MCCB14
80-071/ 03L	09/14/80	10/14/80	'A' SBGT FAN TRIPPED

REVIEWED BY OMPA LOEB: T. CINTULA **DATE:** 12·10·80

REVIEWED BY NRR DL: John N. Hannon **DATE:** 12-11-80

PILGRIM

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-293
2. Utility Contact: G. G. Whitney (617) 746-7900
3. Licensed Thermal Power (MWt): 1998
4. Nameplate Rating (Gross MWe): 780 x 0.87 = 678
5. Design Electrical Rating (Net MWe): 655
6. Maximum Dependable Capacity (Gross MWe): 690
7. Maximum Dependable Capacity (Net MWe): 670
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	745.0	7,320.0	69,216.0
12. Hours Reactor Was Critical	599.1	3,855.9	48,836.8
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	581.0	3,694.8	47,247.3
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,117,080	6,734,016	79,825,128
17. Gross Electrical Energy (MWH)	388,440	2,306,010	26,470,844
18. Net Electrical Energy (MWH)	373,919	2,217,942	25,424,065
19. Unit Service Factor	78.0	50.5	68.3
20. Unit Availability Factor	78.0	50.5	68.3
21. Unit Capacity Factor (MDC Net)	74.9	45.2	54.8
22. Unit Capacity Factor (DER Net)	76.6	46.3	56.1
23. Unit Forced Outage Rate	22.0	6.7	10.3

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 01/02/80.

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
9	10/01/80	F	118.6	A	3		IA		AIR BUBBLE IN FEEDWATER SYSTEM CAUSED HIGH RADIATION TRIP. MONITOR TRIP WAS FOUND TO BE SET TOO LOW. MONITOR WAS RECALIBRATED.
10	10/07/80	F	33.3	A	1	80-69/01T-0	SF	VALVEX	HIGH N2 PRESSURE TO SV CAUSED SRV TO LIFT. PRESSURE REDUCED.
11	10/31/80	F	12.1	A	1	80-80/01X-0	SF	VALVEX	SAME CAUSE AND CORRECTIVE ACTION AS OUTAGE 10.

SUMMARY

PILGRIM 1 OPERATED NORMALLY DURING OCTOBER, WITH THREE OUTAGES DUE TO EQUIPMENT FAILURE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

POINT BEACH 1

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Wisconsin/Manitowoc 15 miles N of Manitowoc
2.	Electric Reliability Council	Mid-America Interpool Network
3.	Type of Reactor	PWR
4.	Power Level (MWT)	1518
5.	Design Electrical Rating (MWE Net)	497
6.	Date of Initial Criticality	November 2, 1970
7.	Date of Commercial Operation	December 21, 1970
8.	Condenser Cooling Method	Straight Through
9.	Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10.	Licensee	Wisconsin Electric Power Co.
11.	Corporate Address	231 West Michigan Milwaukee, Wisconsin
12.	Corporate Contact	S. Burstein
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	R. F. Warnick
19.	IE Resident Inspector	W. Guldemond
20.	Licensing Project Manager	C. M. Trammell
21.	Docket Number	50-266
22.	License Number/Date of Issue	DPR-24 issued 10/05/70
23.	Public Document Room	Joseph Mann Public Library Two Rivers, Wisconsin Manitowoc Public Library Manitowoc, Wisconsin (Temporary LPDR for spent fuel pool capacity increase amendment only

INSPECTION STATUS

ENFORCEMENT STATUS

10 CFR 50.55A SECTION G(4) REQUIRES COMPONENTS WHICH ARE CLASSIFIED AS ASME CODE CLASS 1, CLASS 2 AND CLASS 3 TO MEET THE INSERVICE INSPECTION REQUIREMENTS SET FORTH IN SECTION XI OF EDITIONS OF THE ASME BOILER AND PRESSURE VESSEL CODE AND ADDENDA. THE 1974 EDITION OF SECTION XI, SUBSECTION IWV, ARTICLE IWV-3000, PARAGRAPH IWV-3410C(3) REQUIRES IN PART THAT, "IF AN INCREASE IN STROKE TIME OF 25% OR MORE FROM THE PREVIOUS TEST FOR VALVES WITH STROKE TIMES GREATER THAN 10 SECONDS OR 50% OR MORE FOR VALVES WITH STROKE TIMES LESS THAN OR EQUAL TO 10 SECONDS IS OBSERVED, TEST FREQUENCY SHALL BE INCREASED TO ONCE EACH MONTH UNTIL CORRECTIVE ACTION IS TAKEN..." CONTRARY TO THE ABOVE, A PARITAL REVIEW OF DATA FROM IT-20, AND IT-45 FOR THE PERIOD 4-77 TO PRESENT, REVEALED THAT VALVES 539, 508, 850B, 538, AND 897A SHOWED INCREASES IN CYCLE TIMES AS LISTED BELOW AND WERE NOT SUBJECTED TO SHORTENED TEST INTERVALS. VALVE NUMBER 539, TEST INTERVAL 10/19/79-01/18/80, STROKE TIME PERCENTAGE INCREASE 97, STROKE DIRECTION OPEN; VALVE NUMBER 508, TEST INTERVAL 07/21/79-10/19/79, STROKE TIME PERCENTAGE INCREASE 94, STROKE DIRECTION OPEN, TEST INTERVAL 10/19/79-01/18/80, STROKE TIME PERCENTAGE INCREASE 151, STROKE DIRECTION CLOSED, TEST INTERVAL 04/18/80-07/19/80, STROKE TIME PERCENTAGE INCREASE 135, STROKE DIRECTION OPEN; VALVE NUMBER 538, TEST INTERVAL 10/24/77-01/23/78, STROKE TIME PERCENTAGE INCREASE 53, STROKE DIRECTION OPEN; VALVE NUMBER 897A, TEST INTERVAL 01/05/80-04/07/80, STROKE TIME PERCENTAGE INCREASE 67, STROKE DIRECTION CLOSED; VALVE NUMBER 850B, TEST INTERVAL 01/19/80-05/11/80, STROKE TIME PERCENTAGE INCREASE 58, STROKE DIRECTION OPEN.
(8014 INFRACTION)

TECHNICAL SPECIFICATION 15.3.3.B.2.C REQUIRES FOR THE CONTAINMENT SPRAY SYSTEM THAT, "ANY VALVE REQUIRED FOR THE FUNCTIONING OF THE SYSTEM DURING ACCIDENT CONDITIONS MAY BE INOPERABLE PROVIDED REPAIRS ARE COMPLETED WITHIN 24 HOURS. PRIOR TO INITIATING REPAIRS, ALL VALVES IN THE SYSTEM THAT PROVIDE THE DUPLICATE FUNCTION SHALL BE TESTED TO DEMONSTRATE OPERABILITY." CONTRARY TO THE ABOVE, ON AUGUST 27, 1980 AFTER COMPLETING PACKING ADJUSTMENT ON THE ISOLATION VALVE FROM THE RHR SYSTEM TO A UNIT 1 CONTAINMENT SPRAY PUMP (817A), BUT PRIOR TO PERFORMING POST MAINTENANCE TESTING ON THE VALVE, PACKING ADJUSTMENT WAS INITIATED ON THE DUPLICATE VALVE (871B) TO THE OTHER SPRAY PUMP.
(8014 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENTS

- THE UNIT 1 STEAM GENERATOR TESTING REQUIRED BY THE NOVEMBER 30, 1979 CONFIRMATORY ORDER AS AMENDED WAS ACCOMPLISHED JULY 26 THROUGH AUGUST 9, 1980. (80-14)

FACILITY ITEMS

- MODIFICATIONS COMMENCED DURING THE WEEK OF JULY 14 TO THE CIRCULATING WATER INTAKE STRUCTURE TO ALLEVIATE ICING PROBLEMS ARE CONTINUING. (80-14)
- MODIFICATIONS TO THE CIRCULATING WATER INTAKE STRUCTURE ARE ESSENTIALLY COMPLETE. THE MODIFICATIONS WERE MADE TO ALLEVIATE A WINTER FREEZING PROBLEM. (80-15)
- CONSTRUCTION HAS COMMENCED ON THE ON-SITE TECHNICIAN SUPPORT CENTER. THE EFFORT REQUIRED REROUTING OF UNDERGROUND FUEL LINES TO THE EMERGENCY DIESEL GENERATORS. (80-15)

PLANT STATUS

THE UNIT OPERATED ROUTINELY DURING THE MONTH.

- LAST IE SITE INSPECTION DATE: SEPTEMBER 2-30, 1980
- INSPECTION REPORT NO: 80-15

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-10/ 01L-0	08/27/80	09/18/80	FAILURE TO PERFORM PROPER POST MAINTENANCE TESTING ON CSSV.
80-11/ 03L-0	09/26/80	10/06/80	FROM AUGUST THROUGH SEPTEMBER CSSS SPRAY ADDITIVE VALVES TESTED WITH SATISFACTORY RESULTS.

SUMMARY

INSPECTION SUMMARY

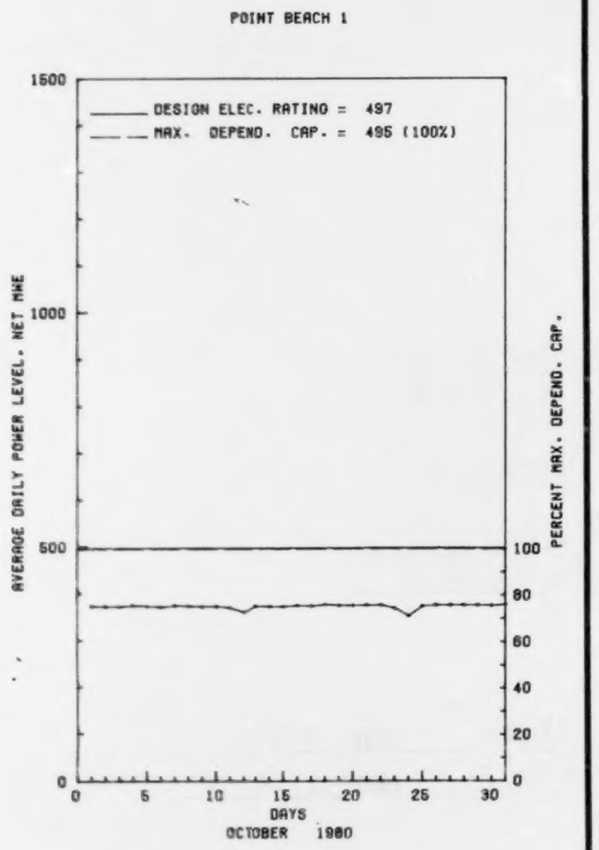
- INSPECTION ON AUGUST 1, 4-8, 11-15, 18-22 AND 25-29, (80-14): ROUTINE, RESIDENT INSPECTION OF OPERATIONAL SAFETY VERIFICATION, MONTHLY MAINTENANCE OBSERVATION, MONTHLY SURVEILLANCE OBSERVATION, LICENSEE EVENT REPORTS FOLLOWUP, IE BULLETIN FOLLOWUP, IE CIRCULAR FOLLOWUP, LICENSEE PLANS FOR COPING WITH STRIKES, FOLLOWUP ON HEADQUARTERS REQUESTS, FOLLOWUP ON ITEMS OF NONCOMPLIANCE, AND INDEPENDENT INSPECTION OF INSERVICE INSPECTION PROGRAM. THE INSPECTION INVOLVED 192 INSPECTOR-HOURS ONSITE BY THREE NRC INSPECTORS INCLUDING 12 INSPECTOR-HOURS ONSITE DURING OFF-SHIFTS. OF THE 10 AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED IN EIGHT AREAS. TWO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED, ONE IN EACH OF TWO AREAS (FAILURE TO COMPLY WITH ASME SECTION XI CODE REQUIREMENTS FOR INSERVICE TESTING OF VALVES; AND EXCEEDING TECHNICAL SPECIFICATION REQUIREMENTS DUE TO IMPROPER VALVE TESTING SEQUENCE ON THE CONTAINMENT SPRAY SYSTEM).
- INSPECTION ON SEPTEMBER 2-5, 8-12, 15-19, 22-26, AND 29-30, (80-15): ROUTINE RESIDENT INSPECTION OF OPERATIONAL SAFETY VERIFICATION, MONTHLY MAINTENANCE OBSERVATION, MONTHLY SURVEILLANCE OBSERVATION, FOLLOWUP ON LICENSEE EVENT REPORTS, IE BULLETIN AND IE CIRCULAR FOLLOWUP, REVIEW OF PLANT OPERATIONS, FOLLOWUP ON ITEMS OF NONCOMPLIANCE, AND RECEIPT OF NEW FUEL. THE INSPECTION INVOLVED 197 INSPECTOR-HOURS ONSITE BY TWO NRC INSPECTORS INCLUDING 50 INSPECTOR-HOURS DURING OFF-SHIFTS. NO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED.

REVIEWED BY OMPA LOEB: P E Bobe **DATE:** 12/10/80

REVIEWED BY NRR DL: **DATE:** 12/10/80

POINT BEACH 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-266
2. Utility Contact: C.W. FAY (414) 277-2811
3. Licensed Thermal Power (MWt): 1518
4. Nameplate Rating (Gross MWe): 582 x 0.9 = 524
5. Design Electrical Rating (Net MWe): 497
6. Maximum Dependable Capacity (Gross MWe): 519
7. Maximum Dependable Capacity (Net MWe): 495
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): 390
10. Reasons for Restrictions, If Any:
INHIBIT CORROSION OF STEAM GENERATOR TUBES.

ITEM 21 CALCULATED BY USING A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	745.0	7,320.0	87,552.0
12. Hours Reactor Was Critical	745.0	6,225.8	72,742.3
13. Reactor Reserve Shutdown Hours	.0	19.5	631.7
14. Hours Generator On-Line	745.0	6,075.2	70,469.2
15. Unit Reserve Shutdown Hours	.0	152.8	755.2
16. Gross Thermal Energy (MWH)	901,880	7,331,099	98,083,894
17. Gross Electrical Energy (MWH)	293,160	2,381,960	33,015,710
18. Net Electrical Energy (MWH)	277,200	2,249,770	31,439,948
19. Unit Service Factor	100.0	83.0	80.5
20. Unit Availability Factor	100.0	85.1	81.4
*21. Unit Capacity Factor (MDC Net)	75.2	62.1	73.7
22. Unit Capacity Factor (DER Net)	74.9	61.8	72.3
23. Unit Forced Outage Rate	.0	.1	3.4

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
REFUELING - 11/28/80 - 5 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	NONE								

SUMMARY

POINT BEACH 1 OPERATED UNDER A UTILITY IMPOSED RESTRICTION, WITH NO OTHER REDUCTIONS OR OUTAGES REPORTED DURING OCTOBER.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

POINT BEACH 2

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Wisconsin/Manitowoc 15 miles N of Manitowoc
2.	Electric Reliability Council	Mid-America Interpool Network
3.	Type of Reactor	PWR
4.	Power Level (MWT)	1518
5.	Design Electrical Rating (MWE Net)	497
6.	Date of Initial Criticality	May 30, 1972
7.	Date of Commercial Operation	October 1, 1972
8.	Condenser Cooling Method	Straight Through
9.	Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10.	Licensee	Wisconsin Electric Power Co.
11.	Corporate Address	231 West Michigan Milwaukee, Wisconsin
12.	Corporate Contact	S. Burstein
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	R. F. Warnick
19.	IE Resident Inspector	W. Guldemond
20.	Licensing Project Manager	C. M. Trammell
21.	Docket Number	50-301
22.	License Number/Date of Issue	DPR-27 issued 11/16/71
23.	Public Document Room	Joseph Mann Public Library Two Rivers, Wisconsin Manitowoc Public Library Manitowoc, Wisconsin (Temporary LPDR for spent fuel pool capacity increase amendment only

INSPECTION STATUS

ENFORCEMENT STATUS

10 CFR 50.55A SECTION G(4) REQUIRES COMPONENTS WHICH ARE CLASSIFIED AS ASME CODE CLASS 1, CLASS 2 AND CLASS 3 TO MEET THE INSERVICE INSPECTION REQUIREMENTS SET FORTH IN SECTION XI OF EDITIONS OF THE ASME BOILER AND PRESSURE VESSEL CODE AND ADDENDA. THE 1974 EDITION OF SECTION XI, SUBSECTION IWV, ARTICLE IWV-3000, PARAGRAPH IWV-3410C(3) REQUIRES IN PART THAT, "IF AN INCREASE IN STROKE TIME OF 25% OR MORE FROM THE PREVIOUS TEST FOR VALVES WITH STROKE TIMES GREATER THAN 10 SECONDS OR 50% OR MORE FOR VALVES WITH STROKE TIMES LESS THAN OR EQUAL TO 10 SECONDS IS OBSERVED, TEST FREQUENCY SHALL BE INCREASED TO ONCE EACH MONTH UNTIL CORRECTIVE ACTION IS TAKEN..." CONTRARY TO THE ABOVE, A PARITAL REVIEW OF DATA FROM IT-20, AND IT-45 FOR THE PERIOD 4-77 TO PRESENT, REVEALED THAT VALVES 539, 508, 850B, 538, AND 897A SHOWED INCREASES IN CYCLE TIMES AS LISTED BELOW AND WERE NOT SUBJECTED TO SHORTENED TEST INTERVALS. VALVE NUMBER 539, TEST INTERVAL 10/19/79-01/18/80, STROKE TIME PERCENTAGE INCREASE 97, STROKE DIRECTION OPEN; VALVE NUMBER 508, TEST INTERVAL 07/21/79-10/19/79, STROKE TIME PERCENTAGE INCREASE 94, STROKE DIRECTION OPEN, TEST INTERVAL 10/19/79-01/18/80, STROKE TIME PERCENTAGE INCREASE 151, STROKE DIRECTION CLOSED, TEST INTERVAL 04/18/80-07/19/80, STROKE TIME PERCENTAGE INCREASE 135, STROKE DIRECTION OPEN; VALVE NUMBER 538, TEST INTERVAL 10/24/77-01/23/78, STROKE TIME PERCENTAGE INCREASE 53, STROKE DIRECTION OPEN; VALVE NUMBER 897A, TEST INTERVAL 01/05/80-04/07/80, STROKE TIME PERCENTAGE INCREASE 67, STROKE DIRECTION CLOSED; VALVE NUMBER 850B, TEST INTERVAL 01/19/80-05/11/80, STROKE TIME PERCENTAGE INCREASE 58, STROKE DIRECTION OPEN.
(8014 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

FACILITY ITEMS

- MODIFICATIONS COMMENCED DURING THE WEEK OF JULY 14 TO THE CIRCULATING WATER INTAKE STRUCTURE TO ALLEVIATE ICING PROBLEMS ARE CONTINUING. (80-14)
- MODIFICATIONS TO THE CIRCULATING WATER INTAKE STRUCTURE ARE ESSENTIALLY COMPLETE. THE MODIFICATIONS WERE MADE TO ALLEVIATE A WINTER FREEZING PROBLEM. (80-15)
- CONSTRUCTION HAS COMMENCED ON THE ON-SITE TECHNICIAN SUPPORT CENTER. THE EFFORT REQUIRED REROUTING OF UNDERGROUND FUEL LINES TO THE EMERGENCY DIESEL GENERATORS. (80-15)

PLANT STATUS

THE UNIT OPERATED ROUTINELY DURING THE MONTH.

+LAST IE SITE INSPECTION DATE: SEPTEMBER 2-30, 1980

+ INSPECTION REPORT NO: 80-15

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-08/01T-0	09/12/80	09/26/80	FIALURE OF MSIV TO SHUT IN RESPONSE TO SI.
80-11/03L-0	09/26/80	10/06/80	FROM AUGUST THROUGH SEPTEMBER CSSS SPRAY ADDITIVE VALVES TESTED WITH SATISFACTORY RESULTS.

==

SUMMARY

INSPECTION SUMMARY

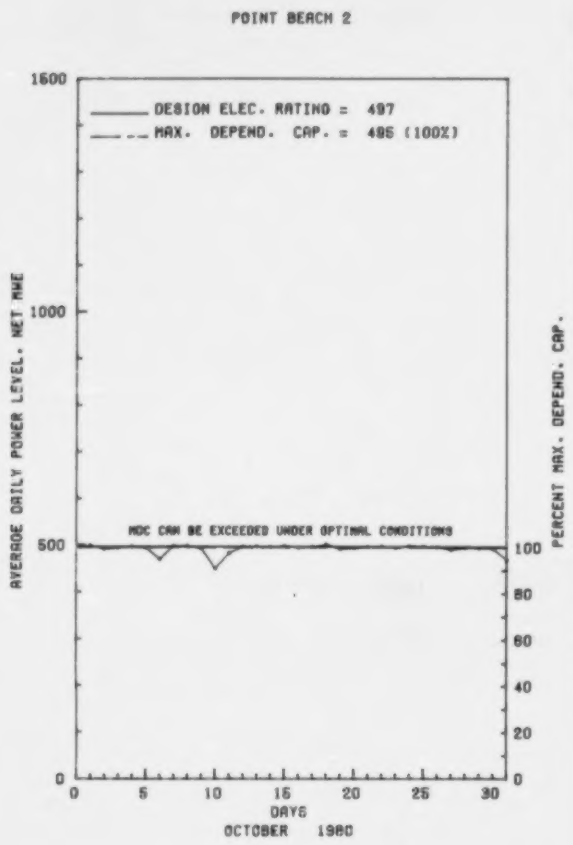
- INSPECTION ON AUGUST 1, 4-8, 11-15, 18-22 AND 25-29, (80-14): ROUTINE, RESIDENT INSPECTION OF OPERATIONAL SAFETY VERIFICATION, MONTHLY MAINTENANCE OBSERVATION, MONTHLY SURVEILLANCE OBSERVATION, LICENSEE EVENT REPORTS FOLLOWUP, IE BULLETIN FOLLOWUP, IE CIRCULAR FOLLOWUP, LICENSEE PLANS FOR COPING WITH STRIKES, FOLLOWUP ON HEADQUARTERS REQUESTS, FOLLOWUP ON ITEMS OF NONCOMPLIANCE, AND INDEPENDENT INSPECTION OF INSERVICE INSPECTION PROGRAM. THE INSPECTION INVOLVED 192 INSPECTOR-HOURS ONSITE BY THREE NRC INSPECTORS INCLUDING 12 INSPECTOR-HOURS ONSITE DURING OFF-SHIFTS. OF THE 10 AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED IN EIGHT AREAS. TWO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED, ONE IN EACH OF TWO AREAS (FAILURE TO COMPLY WITH ASME SECTION XI CODE REQUIREMENTS FOR INSERVICE TESTING OF VALVES; AND EXCEEDING TECHNICAL SPECIFICATION REQUIREMENTS DUE TO IMPROPER VALVE TESTING SEQUENCE ON THE CONTAINMENT SPRAY SYSTEM).
- INSPECTION ON SEPTEMBER 2-5, 8-12, 15-19, 22-26, AND 29-30, (80-15): ROUTINE RESIDENT INSPECTION OF OPERATIONAL SAFETY VERIFICATION, MONTHLY MAINTENANCE OBSERVATION, MONTHLY SURVEILLANCE OBSERVATION, FOLLOWUP ON LICENSEE EVENT REPORTS, IE BULLETIN AND IE CIRCULAR FOLLOWUP, REVIEW OF PLANT OPERATIONS, FOLLOWUP ON ITEMS OF NONCOMPLIANCE, AND PLANT TRIPS. THE INSPECTION INVOLVED 197 INSPECTOR-HOURS ONSITE BY TWO NRC INSPECTORS INCLUDING 50 INSPECTOR-HOURS DURING OFF-SHIFTS. NO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED.

REVIEWED BY OMPA LOEB: DATE: 12/10/80

REVIEWED BY NRR DL: DATE: 12/10/80

POINT BEACH 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-301
2. Utility Contact: C.H. FAY (414) 277-2811
3. Licensed Thermal Power (MWt): 1518
4. Nameplate Rating (Gross MWe): 582 x 0.9 = 524
5. Design Electrical Rating (Net MWe): 497
6. Maximum Dependable Capacity (Gross MWe): 519
7. Maximum Dependable Capacity (Net MWe): 495
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

ITEM 21 CALCULATED BY USING A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	745.0	7,320.0	72,337.0
12. Hours Reactor Was Critical	745.0	6,211.1	65,108.0
13. Reactor Reserve Shutdown Hours	.0	19.6	185.7
14. Hours Generator On-Line	745.0	6,143.5	63,870.5
15. Unit Reserve Shutdown Hours	.0	15.3	121.7
16. Gross Thermal Energy (MWH)	1,112,514	9,033,479	86,686,015
17. Gross Electrical Energy (MWH)	383,910	3,045,380	29,406,720
18. Net Electrical Energy (MWH)	366,708	2,901,813	27,976,940
19. Unit Service Factor	100.0	83.9	88.3
20. Unit Availability Factor	100.0	84.1	88.5
*21. Unit Capacity Factor (MDC Net)	99.4	80.1	78.8
22. Unit Capacity Factor (DER Net)	99.0	79.8	77.8
23. Unit Forced Outage Rate	.0	5.6	1.9

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUELING - 04/17/81 - 5 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	NONE								

SUMMARY

POINT BEACH 2 OPERATED AT OR NEAR FULL POWER DURING OCTOBER, WITH NO REDUCTIONS OR OUTAGES REPORTED.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

PRAIRIE ISLAND 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Minnesota/Goodhue 28 miles SE of Minneapolis
2.	Electric Reliability Council	Mid-Continent Area Reliability Coordination Agreement
3.	Type of Reactor	PWR
4.	Power Level (MWT)	1650
5.	Design Electrical Rating (MWE Net)	530
6.	Date of Initial Criticality	December 1, 1973
7.	Date of Commercial Operation	December 16, 1973
8.	Condenser Cooling Method	Cooling Towers
9.	Condenser Cooling Water Source	Mississippi River

Utility & Contractor Information

10.	Licensee	Northern States Power Company
11.	Corporate Address	414 Nicollet Mall - 8th Floor Minneapolis, Minnesota 55401
12.	Corporate Contact	G. H. Neils (Oper.) L. O. Mayer (Lic.)
13.	Architect/Engineer	Pioneer Services & Eng. Co.
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Northern States Power Co.

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	W. S. Little
19.	IE Resident Inspector	C. D. Feierabend 612-388-8209
20.	Licensing Project Manager	R. E. Martin
21.	Docket Number	50-282
22.	License Number/Date of Issue	DPR-42 issued 8/9/73
23.	Public Document Room	Environmental Conservation Library Minneapolis Public Library 300 Nicollet Mall Minneapolis, Minnesota 55401

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

```
    INSPECTION STATUS

    OTHER ITEMS OF INTEREST
    PLANT STATUS
*   OPERATING NORMALLY.
 LAST IE SITE INSPECTION DATE:  AUGUST 1-31, 1980
        INSPECTION REPORT NO:  80-14
    SUMMARY
    INSPECTION SUMMARY
*   NO INSPECTIONS CONDUCTED.
```

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

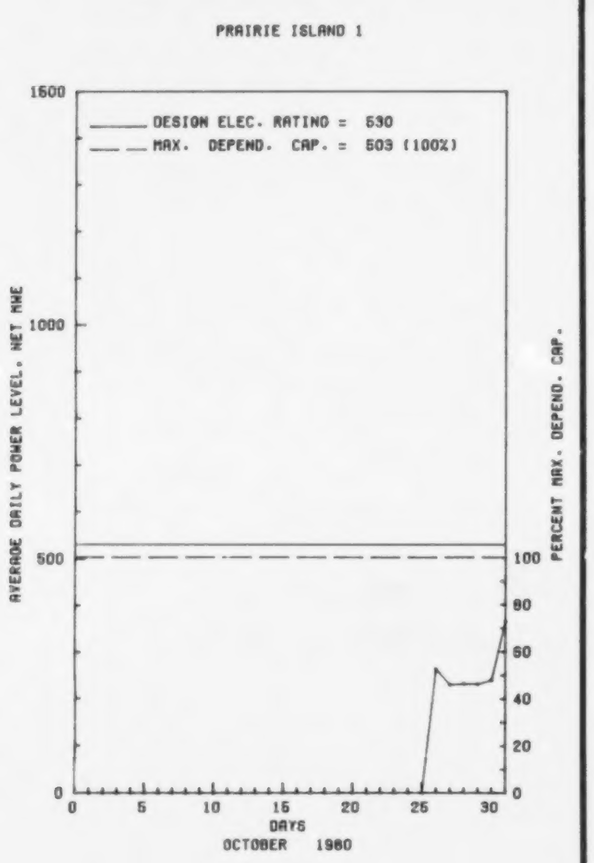
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-16/ 03L-0	05/29/80	06/27/80	WHILE PERFORMING AN INSTRUMENT CALIBRATION AN INSTRUMENT WAS DROPPED ON TRIP THROTTLE.
80-26/ 03L-0	09/16/80	10/15/80	INOPERABILITY OF NO. 22 AUXILIARY FEEDWATER PUMP.

REVIEWED BY OMPA LOEB: P E Bole **DATE:** 12/10/80

REVIEWED BY NRR DL: R E Martin **DATE:** 12/11/80

PRAIRIE ISLAND 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-282
2. Utility Contact: A. A. Hunstad (612) 388-4767
3. Licensed Thermal Power (MWt): 1650
4. Nameplate Rating (Gross MWe): 659 x 0.9 = 593
5. Design Electrical Rating (Net MWe): 530
6. Maximum Dependable Capacity (Gross MWe): 534
7. Maximum Dependable Capacity (Net MWe): 503
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	745.0	7,320.0	60,288.0
12. Hours Reactor Was Critical	244.5	5,500.9	46,709.3
13. Reactor Reserve Shutdown Hours	.0	.0	5,531.9
14. Hours Generator On-Line	164.7	5,411.2	45,534.8
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	133,518	8,274,267	70,353,261
17. Gross Electrical Energy (MWH)	42,740	2,536,420	22,690,090
18. Net Electrical Energy (MWH)	33,922	2,360,986	21,200,238
19. Unit Service Factor	22.1	73.9	75.5
20. Unit Availability Factor	22.1	73.9	75.5
21. Unit Capacity Factor (MDC Net)	9.1	64.1	69.9
22. Unit Capacity Factor (DER Net)	8.6	60.9	66.3
23. Unit Forced Outage Rate	.0	3.2	12.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
REFUELING, SEPTEMBER, 1981, - 6 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	08/31/80	S	564.8	C	4				REFUELING OUTAGE CONTINUES.
	10/24/80	S	15.5	B	9				SHUTDOWN TO PERFORM TURBINE OVERSPEED TEST AND GENERATOR SHORT CIRCUIT TEST.

SUMMARY

PRAIRIE ISLAND 1 RETURNED ON-LINE FROM REFUELING AND TESTING ON OCTOBER 26TH AND REMAINED ON-LINE THRU THE 31ST.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

PRAIRIE ISLAND 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Minnesota/Goodhue 28 miles SE of Minneapolis
2.	Electric Reliability Council	Mid-Continent Area Reliability Coordination Agreement
3.	Type of Reactor	PWR
4.	Power Level (MWT)	1650
5.	Design Electrical Rating (MWE Net)	530
6.	Date of Initial Criticality	December 17, 1974
7.	Date of Commercial Operation	December 21, 1974
8.	Condenser Cooling Method	Cooling Towers
9.	Condenser Cooling Water Source	Mississippi River

Utility & Contractor Information

10.	Licensee	Northern States Power Company
11.	Corporate Address	414 Nicollet Mall - 8th Floor Minneapolis, Minnesota 55401
12.	Corporate Contact	G. H. Neils (Oper.) L. O. Mayer (Lic.)
13.	Architect/Engineer	Pioneer Services & Eng. Co.
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Northern States Power Co.

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	W. S. Little
19.	IE Resident Inspector	C. D. Feierabend 612-388-8209
20.	Licensing Project Manager	R. Martin
21.	Docket Number	50-306
22.	License Number/Date of Issue	DPR-60 issued 10/29/74
23.	Public Document Room	Environmental Conservation Library Minneapolis Public Library 300 Nicollet Mall Minneapolis, Minnesota 55401

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

```
   INSPECTION  STATUS

   OTHER ITEMS OF INTEREST
   PLANT STATUS
   OPERATING ROUTINELY.
LAST IE SITE INSPECTION DATE:  AUGUST 1-31, 1980
      INSPECTION REPORT NO:  80-15
   SUMMARY
   INSPECTION SUMMARY
+  NO INSPECTIONS CONDUCTED.
```

REPORTS RECEIVED FROM LICENSEE

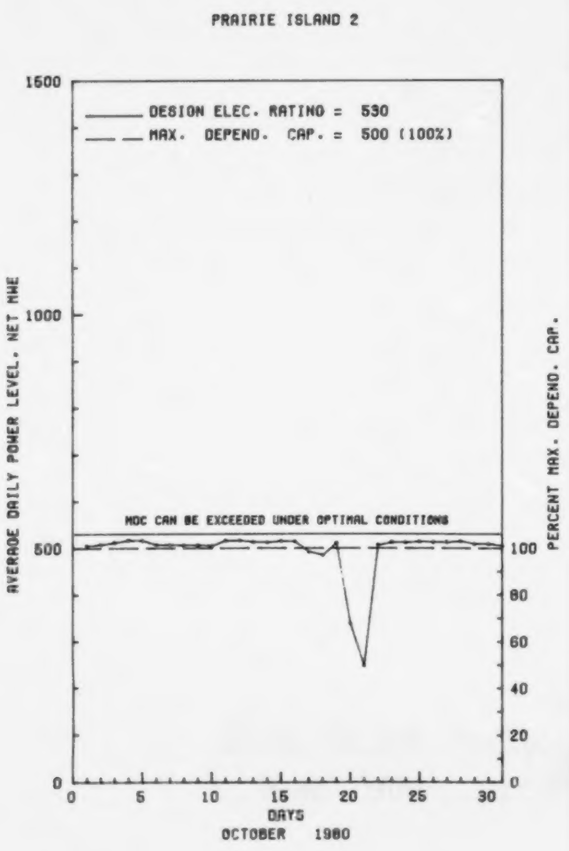
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE			

REVIEWED BY OMPA LOEB: R Bobe **DATE:** 12/10/80

REVIEWED BY NRR DL: RE Martin **DATE:** 12/11/80

PRAIRIE ISLAND 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-306
2. Utility Contact: A. A. Hunstad (612) 388-4767
3. Licensed Thermal Power (MWt): 1650
4. Nameplate Rating (Gross MWe): 659 x 0.9 = 593
5. Design Electrical Rating (Net MWe): 530
6. Maximum Dependable Capacity (Gross MWe): 531
7. Maximum Dependable Capacity (Net MWe): 500
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	745.0	7,320.0	51,406.0
12. Hours Reactor Was Critical	732.4	5,788.8	44,917.8
13. Reactor Reserve Shutdown Hours	12.6	102.0	1,508.0
14. Hours Generator On-Line	727.5	5,705.9	44,106.9
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,184,220	9,054,612	68,727,115
17. Gross Electrical Energy (MWH)	391,690	2,927,420	21,981,550
18. Net Electrical Energy (MWH)	369,102	2,737,844	20,576,476
19. Unit Service Factor	97.7	77.9	85.8
20. Unit Availability Factor	97.7	77.9	85.8
21. Unit Capacity Factor (MDC Net)	99.1	74.8	80.1
22. Unit Capacity Factor (DER Net)	93.5	70.6	75.5
23. Unit Forced Outage Rate	2.3	.7	4.3

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUELING, FEBRUARY, 1981, 6 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	10/20/80	F	17.5	G	2		HA	CKTBRK	CONSTRUCTION PERSON ACCIDENTALLY BUMPED THE TRIP SWITCH FOR THE BREAKER SUPPLYING POWER TO THE TURBINE E-H PUMPS. TURBINE CONTROL VALVES DRIFTED CLOSED ON LOSS OF E-H FLUID PRESSURE. REACTOR WAS TRIPPED MANUALLY.

SUMMARY

PRAIRIE ISLAND 2 OPERATED AT OR NEAR FULL POWER FOR OCTOBER, WITH ONE OUTAGE ON THE 20TH DUE TO OPERATIONAL ERROR.

LEGEND

[1] Type:
F-Forced
S-Scheduled

[2] Reason:
A-Equipment Failure
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error
H-Other

[3] Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Continuations
5-Load Reduction
9-Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

QUAD CITIES 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Illinois/Rock Island 20 miles NE of Moline
2.	Electric Reliability Council	Mid-America Interpool Network
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2511
5.	Design Electrical Rating (MWE Net)	789
6.	Date of Initial Criticality	October 18, 1971
7.	Date of Commercial Operation	February 18, 1973
8.	Condenser Cooling Method	Spray Canal
9.	Condenser Cooling Water Source	Mississippi River

Utility & Contractor Information

10.	Licensee	Commonwealth Edison Co.
11.	Corporate Address	P. O. Box 767 Chicago, Illinois 60690
12.	Corporate Contact	Cordell Reed, Vice President
13.	Architect/Engineer	Sargent and Lundy
14.	Nuclear Steam Supply System	GE
15.	Constructor	United Engineers & Constructors, Inc.

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	R. Spessard
19.	IE Resident Inspector	N. J. Chrissotimos (309) 654-2227
20.	Licensing Project Manager	R. B. Bevan
21.	Docket Number	50-254
22.	License Number/Date of Issue	DPR-29 issued 10/1/71
23.	Public Document Room	Moline Public Library 504 17th Street Moline, Illinois 61265

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENTS

- LICENSEE RECENTLY MODIFIED A FORMER UREA FORMALDEHYDE SOLIDIFICATION SYSTEM (NEVER OPERABLE) TO A DOW SOLIDIFICATION SYSTEM UTILIZING 50 CUBIC FOOT LINERS. SYSTEM STILL IN TESTING PHASE AND ARE EXPERIENCING SOME DIFFICULTY IN SOLIDIFICATION ON FULL SCALE BATCH. (80-24)

FACILITY ITEMS

- UNIT 1 WAS SHUTDOWN ON AUGUST 31, 1980 TO BEGIN A SCHEDULED REFUEL/MAINTENANCE OUTAGE. MAJOR TASKS INCLUDE TURBINE INSPECTION, TORUS MODIFICATIONS CORE SPRAY AND FEEDWATER SPARGER INSPECTIONS. (80-21)

PLANT STATUS

THE UNIT WAS SHUT DOWN ON AUGUST 31, 1980 TO BEGIN A SCHEDULED 12 WEEK REFUELING OUTAGE.

+LAST IE SITE INSPECTION DATE: SEPTEMBER 9-12, 1980

+ INSPECTION REPORT NO: 80-24

SUMMARY

INSPECTION SUMMARY

- INSPECTION ON JULY 21 THROUGH AUGUST 29, (80-19): ROUTINE, UNANNOUNCED RESIDENT INSPECTION OF LICENSEE ACTION ON PREVIOUS INSPECTION FINDINGS, MAINTENANCE OBSERVATION, SURVEILLANCE OBSERVATION, OPERATIONAL SAFETY VERIFICATION, CALIBRATION, QUALITY ASSURANCE PROGRAM ORGANIZATION AND ADMINISTRATION, REVIEW OF PLANT OPERATIONS, FIRE PROTECTION, PARTICIPATION IN NRR/LICENSEE MEETING, REVIEW OF PERIODIC AND SPECIAL REPORTS, FOLLOWUP ON PART 21 REPORT, PARTICIPATION IN INSERVICE INSPECTION MEETING, LICENSEE EVENT REPORT FOLLOWUP, IE CIRCULAR FOLLOWUP, IE BULLETING FOLLOWUP, FOLLOWUP ON REGIONAL REQUESTS, MEDIA CONTACTS. THE INSPECTION INVOLVED 252 INSPECTOR-HOURS ONSITE BY TWO NRC INSPECTORS INCLUDING 35 INSPECTOR-HOURS ONSITE DURING OFF-SHIFTS. OF THE 17 AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED IN 16 AREAS; THREE ITEMS OF NONCOMPLIANCE WERE IDENTIFIED IN TWO AREAS (FAILURE TO FOLLOW PROCEDURES).
- HEALTH PHYSICS APPRAISAL ON MAY 5-16, (80-20): SPECIAL, ANNOUNCED APPRAISAL OF HEALTH PHYSICS PROGRAM, INCLUDING ORGANIZATION, MANAGEMENT, TRAINING AND QUALIFICATIONS, EXPOSURE CONTROLS, ACCESS CONTROLS, SURVEILLANCE, INSTRUMENTATION, FACILITIES AND EQUIPMENT, RADIOACTIVE WASTE MANAGEMENT AND ACCIDENT/RECOVERY CAPABILITIES. THE INSPECTION INVOLVED 450 INSPECTOR-HOURS ONSITE BY FIVE NRC INSPECTORS. SIGNIFICANT HEALTH PHYSICS PROGRAM WEAKNESSES WERE IDENTIFIED IN THE AREAS OF TRAINING; EXPOSURE CONTROL AND ALARA; ACCESS CONTROLS; CONTAMINATION CONTROLS; INSTRUMENTATION; AND FACILITIES. TWO APPARENT ITEMS OF NONCOMPLIANCE WERE IDENTIFIED (UNCONTROLLED HIGH RADIATION AREA ACCESS; AND FAILURE TO FOLLOW PROCEDURES).
- INSPECTION ON SEPTEMBER 9-12, (80-24): ROUTINE, UNANNOUNCED INSPECTION OF OPERATIONAL RADIATION PROTECTION PROGRAM DURING REFUELING AND MAINTENANCE, INCLUDING: ORGANIZATION AND STAFFING; TRAINING; RADIATION PROTECTION PROCEDURES; PLANNING, PREPARATION, AND ALARA; EXPOSURE CONTROL; POSTING AND CONTROL; MATERIAL CONTROL; SURVEYS; MISSING FISSION CHAMBER; INDUSTRIAL ACCIDENT; FILM BADGE VERSUS PENCIL DOSIMETER DISCREPANCY; AND IE BULLETIN NO. 80-10. THE INSPECTION INVOLVED 36 INSPECTOR-HOURS ONSITE BY ONE NRC INSPECTOR. NO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED IN ANY OF THE 12 AREAS INSPECTED.

REPORTS RECEIVED FROM LICENSEE

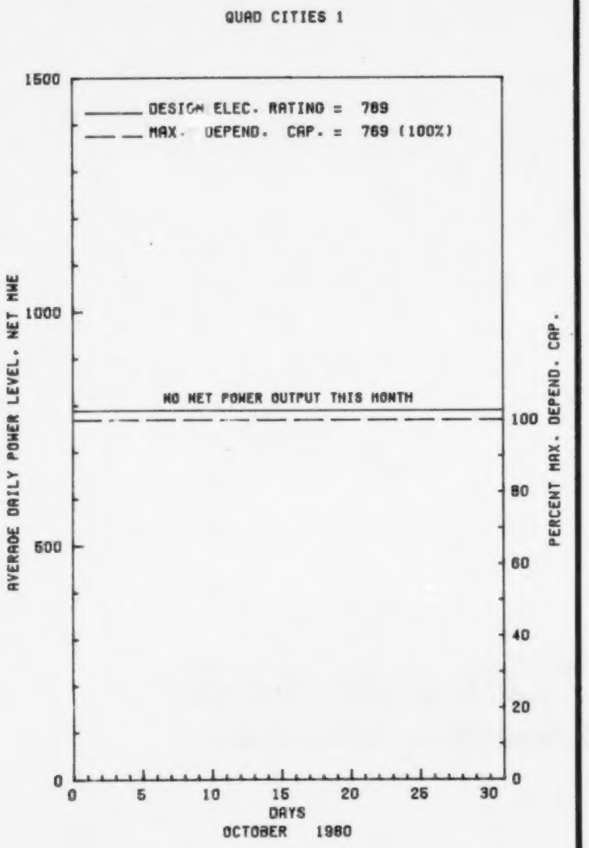
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-20/ 03L-0	08/28/80	09/19/80	NITROGEN MAKEUP VALVE WOULD NOT CLOSE DURING SUPP. CHAMBER TO DRYWELL SEP. TEST.
80-21/ 03L-0	09/01/80	09/19/80	FIVE OF EIGHT MAIN STEAM ISOLATION VALVES FAILED PERIODIC LOCAL LEAK RATE TESTING.
80-22/ 03L-0	09/03/80	09/22/80	ROUTINE SURVEILLANCE TEST DETERMINED THAT ONE REDUNDANT HPCI ROOM HIGH AREA TEMP. SWITCH FOR SYSTEM ISOLATION HAD DRIFTED TO A TRIP POINT OF 212 DEGREES F-LIMIT IS 200 DEGREES F.
80-23/ 03L-0	09/10/80	09/29/80	TWO 4 1/2 INCH FIRE BARRIER PENETRATIONS WERE LEFT UNSEALED AS A RESULT OF CONSTRUCT. PERSONNEL FAILURE TO FOLLOW PROCEDURES.
80-24/ 03L-0	09/14/80	10/08/80	CORE ALTERATION (FUEL REMOVAL) WAS ACCOMPLISHED WITH LESS THAN 3 CPS ON SRM, IN VIOLATION OF T.S.
80-25/ 01T-0	09/13/80	09/23/80	CRACKING OF CORE SPRAY SYSTEM PIPING ELBOWS AT VESSEL SAFE END DETECTED DURING INSERVICE INSPECTIONS.

REVIEWED BY OMPA LOEB: T. Cintula **DATE:** 12·10·80

REVIEWED BY NRR DL: **DATE:** 12/1/80

QUAD CITIES 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-254
2. Utility Contact: R. TUBBS (309) 654-2241 X176
3. Licensed Thermal Power (MWt): 2511
4. Nameplate Rating (Gross MWe): 920 x 0.9 = 828
5. Design Electrical Rating (Net MWe): 789
6. Maximum Dependable Capacity (Gross MWe): 813
7. Maximum Dependable Capacity (Net MWe): 769
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	745.0	7,320.0	74,280.0
12. Hours Reactor Was Critical	.0	5,725.1	69,439.2
13. Reactor Reserve Shutdown Hours	.0	.0	3,421.9
14. Hours Generator On-Line	.0	5,623.1	57,664.0
15. Unit Reserve Shutdown Hours	.0	.0	909.2
16. Gross Thermal Energy (MWH)	0	11,394,718	115,938,699
17. Gross Electrical Energy (MWH)	53	3,662,811	37,285,865
18. Net Electrical Energy (MWH)	-9,563	3,360,883	34,776,419
19. Unit Service Factor	.0	76.8	77.6
20. Unit Availability Factor	.0	76.8	78.9
21. Unit Capacity Factor (MDC Net)	.0	59.7	60.9
22. Unit Capacity Factor (DER Net)	.0	58.2	59.3
23. Unit Forced Outage Rate	.0	3.6	7.4

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 12/04/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
13	08/31/80	S	745.0	C	4		RC	FUELXX	CONTINUATION OF END OF CYCLE FIVE REFUELING OUTAGE.

SUMMARY

QUAD CITIES 1 REMAINED OFF-LINE DURING OCTOBER AS REFUELING OUTAGE CONTINUED.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

QUAD CITIES 2

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Illinois/Rock Island 20 miles NE of Moline
2.	Electric Reliability Council	Mid-America Interpool Network
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2511
5.	Design Electrical Rating (MWE Net)	789
6.	Date of Initial Criticality	April 26, 1972
7.	Date of Commercial Operation	March 10, 1973
8.	Condenser Cooling Method	Spray Canal
9.	Condenser Cooling Water Source	Mississippi River

Utility & Contractor Information

10.	Licensee	Commonwealth Edison Co.
11.	Corporate Address	P. O. Box 767 Chicago, Illinois 60690
12.	Corporate Contact	Cordell Reed, Vice President
13.	Architect/Engineer	Sargent & Lundy
14.	Nuclear Steam Supply System	GE
15.	Constructor	United Engineers & Constructors, Inc.

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	R. Spessard
19.	IE Resident Inspector	N. J. Chrissotimos (309) 654-2227
20.	Licensing Project Manager	R. B. Bevan
21.	Docket Number	50-265
22.	License Number/Date of Issue	DPR-30 issued 3/21/72
23.	Public Document Room	Moline Public Library 504 17th Street Moline, Illinois 61265

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENTS

+ ON AUGUST 30, 1980, UNIT 2 RECIRC MG SET TRIPPED DUE TO A FAILED TRANSFORMER. UNIT LOAD DECREASED FROM 745 MWE TO 340 MWE. RECIRC PUMP WAS RETURNED TO SERVICE APPROXIMATELY 6 HOURS LATER. (80-21)

PLANT STATUS

PLANT OPERATING ROUTINELY.

\+LAST IE SITE INSPECTION DATE: SEPT. 9 - 12, 1980

+ INSPECTION REPORT NO: 80-22

SUMMARY

INSPECTION SUMMARY

+ INSPECTION ON JULY 21 THROUGH AUGUST 29, (80-21): ROUTINE, UNANNOUNCED RESIDENT INSPECTION OF LICENSEE ACTION ON PREVIOUS INSPECTION FINDINGS, MAINTENANCE OBSERVATION, SURVEILLANCE OBSERVATION, OPERATIONAL SAFETY VERIFICATION, CALIBRATION, QUALITY ASSURANCE PROGRAM ORGANIZATION AND ADMINISTRATION, REVIEW OF PLANT OPERATIONS, FIRE PROTECTION, PARTICIPATION IN NRR/LICENSEE MEETING, REVIEW OF PERIODIC AND SPECIAL REPORTS, FOLLOWUP ON PART 21 REPORT, PARTICIPATION IN INSERVICE INSPECTION MEETING, LICENSEE EVENT REPORT FOLLOWUP, IE CIRCULAR FOLLOWUP, IE BULLETING FOLLOWUP, FOLLOWUP ON REGIONAL REQUESTS, MEDIA CONTACTS, AND INOFFICE REVIEW OF THE UNIT 2 REACTOR BUILDING INTEGRATED LEAK RATE TEST REPORT OF MARCH 12, 1980. THE INSPECTION INVOLVED 252 INSPECTOR-HOURS ONSITE BY TWO NRC INSPECTORS INCLUDING 35 INSPECTOR-HOURS ONSITE DURING OFF-SHIFTS. OF THE 18 AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED.

+ HEALTH PHYSICS APPRAISAL ON MAY 5-16, (80-20): SPECIAL, ANNOUNCED APPRAISAL OF HEALTH PHYSICS PROGRAM, INCLUDING ORGANIZATION, MANAGEMENT, TRAINING AND QUALIFICATIONS, EXPOSURE CONTROLS, ACCESS CONTROLS, SURVEILLANCE, INSTRUMENTATION, FACILITIES AND EQUIPMENT, RADIOACTIVE WASTE MANAGEMENT AND ACCIDENT/RECOVERY CAPABILITIES. THE INSPECTION INVOLVED 450 INSPECTOR-HOURS ONSITE BY FIVE NRC INSPECTORS. SIGNIFICANT HEALTH PHYSICS PROGRAM WEAKNESSES WERE IDENTIFIED IN THE AREAS OF TRAINING; EXPOSURE CONTROL AND ALARA; ACCESS CONTROLS; CONTAMINATION CONTROLS; INSTRUMENTATION; AND FACILITIES. TWO APPARENT ITEMS OF NONCOMPLIANCE WERE IDENTIFIED (UNCONTROLLED HIGH RADIATION AREA ACCESS; AND FAILURE TO FOLLOW PROCEDURES).

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

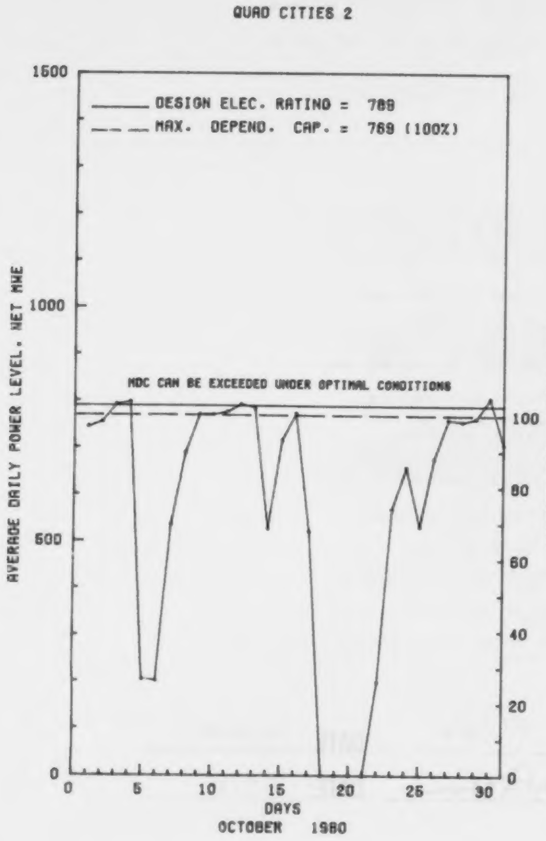
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-21/ 03L-0	09/14/80	10/06/80	INTERMITTENT TORQUE SWITCH FAILURE RENDERED CORE SPRAY VALVE 3A INOPERABLE.

REVIEWED BY OMPA LOEB: T. CINTULA **DATE:** 12.10.80

REVIEWED BY NRR DL: R. Bevan/ **DATE:** 12/8/80

QUAD CITIES 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-265
2. Utility Contact: ROBERT TUBBS (309) 654-2241 X176
3. Licensed Thermal Power (MWt): 2511
4. Nameplate Rating (Gross MWe): 920 x 0.9 = 828
5. Design Electrical Rating (Net MWe): 789
6. Maximum Dependable Capacity (Gross MWe): 813
7. Maximum Dependable Capacity (Net MWe): 769
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	745.0	7,320.0	73,390.0
12. Hours Reactor Was Critical	652.3	6,433.8	57,562.7
13. Reactor Reserve Shutdown Hours	.0	.0	2,985.8
14. Hours Generator On-Line	617.1	6,279.1	55,070.9
15. Unit Reserve Shutdown Hours	.0	.0	702.9
16. Gross Thermal Energy (MWH)	1,367,840	9,546,745	112,562,854
17. Gross Electrical Energy (MWH)	443,666	3,000,736	35,863,786
18. Net Electrical Energy (MWH)	421,353	2,817,269	33,559,794
19. Unit Service Factor	82.8	58.5	75.0
20. Unit Availability Factor	82.8	58.5	76.0
21. Unit Capacity Factor (MDC Net)	73.5	50.0	59.5
22. Unit Capacity Factor (DER Net)	71.7	48.8	58.0
23. Unit Forced Outage Rate	3.3	5.4	9.0

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
15	10/05/80	S	.0	B	5	80-22/03L	CD	VALVEX	POWER REDUCTION TO TIME MAIN STEAM ISOLATION VALVES.
16	10/05/80	F	21.1	H	2		CD	VALVEX	REACTOR MANUALLY SCRAMMED DUE TO FIRE CAUSED BY OIL LEAKING FROM SPEED ADJUSTING VALVE ON MSIV, AND FLASHING TO FIRE WHEN CONTACTING HOT VALVE BODY.
17	10/14/80	F	.0	A	5		SD	INSTRU	POWER REDUCTION TO REPLACE FAULTY TEMPERATURE SWITCH IN PRIMARY CONTAINMENT ISOLATION CIRCUIT.
18	10/17/80	S	106.8	B	1				SCHEDULED OUTAGE TO PERFORM BATTERY TESTS AND MISCELLANEOUS MAINTENANCE ITEMS.

SUMMARY

QUAD CITIES 2 OPERATED WITH TWO OUTAGES AND TWO REDUCTIONS LISTED IN DETAIL ABOVE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

RANCHO SECO

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	California/Sacramento 25 miles SE of Sacramento
2.	Electric Reliability Council	Western Systems Coordinating Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2772
5.	Design Electrical Rating (MWE Net)	918
6.	Date of Initial Criticality	September 16, 1974
7.	Date of Commercial Operation	April 17, 1975 (92.6% of full power)
8.	Condenser Cooling Method	Closed Cycle
9.	Condenser Cooling Water Source	Natural Draft Cooling Towers - Folsom Canal

Utility & Contractor Information

10.	Licensee	Sacramento Municipal Utility District
11.	Corporate Address	6201 South Street P. O. Box 15830 Sacramento, California
12.	Corporate Contact	J. J. Mattimoe Assistant General Manager, Chief Engineer
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	B&W
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	V
17.	IE Branch Chief	J. L. Crews
18.	IE Section Chief	B. H. Faulkenberry
19.	IE Resident Inspector	H. L. Canter (209) 748-2791
20.	Licensing Project Manager	M. L. Padovan
21.	Docket Number	50-312
22.	License Number/Date of Issue	DPR-54, 8/16/74
23.	Public Document Room	Business and Municipal Department Sacramento City-County Library 828 I Street Sacramento, California 95814

INSPECTION STATUS

ENFORCEMENT STATUS

TECHNICAL SPECIFICATION TABLE 2.3-1 SPECIFIES THAT THE NUCLEAR POWER TRIP SETTING IS 5% OF RATED MAXIMUM WHEN THE PLANT IS IN THE SHUTDOWN BYPASS MODE. CONTRARY TO THE REQUIREMENTS, AT 1353 HOURS ON AUGUST 13, 1980, THE PLANT WAS PLACED IN THE SHUTDOWN BYPASS MODE, BUT THE NUCLEAR POWER TRIP SETTING OF 5% OF RATED MAXIMUM WAS NOT INSTITUTED UNTIL 0840 HOURS ON AUGUST 20, 1980.
(8026 INFRACTION)

TECHNICAL SPECIFICATION 6.8.1 STATES IN PART THAT WRITTEN PROCEDURES SHALL BE ESTABLISHED, IMPLEMENTED, AND MAINTAINED COVERING THE ACTIVITIES REFERENCED IN THE APPLICABLE PROCEDURES RECOMMENDED IN APPENDIX A OF REGULATORY GUIDE 1.33, NOVEMBER 1972. APPENDIX A OF REGULATORY GUIDE 1.33, NOVEMBER 1972, STATES IN PART THAT INSTRUCTIONS FOR ENERGIZING, FILLING, VENTING, DRAINING, STARTUP, SHUTDOWN, AND CHANGING MODES OF OPERATION SHOULD BE PREPARED, AS APPROPRIATE, FOR 29 SEPARATE SYSTEMS INCLUDING THE REACTOR CONTROL AND PROTECTION SYSTEM. STEP 5.16 OF STATION PROCEDURE B.4, PLANT SHUTDOWN AND COOLDOWN STATES THAT ONE IS TO "INITIATE REACTOR PROTECTION SYSTEM SHUTDOWN BYPASS AND RESET THE HIGH FLUX TRIP TO 5% AS PER OP A.69, SECTION 5.2." CONTRARY TO THE REQUIREMENTS, ON AUGUST 20, 1980, STEP 5.16 OF STATION PROCEDURE B.4 WAS NOT IMPLEMENTED WHICH RESULTED IN THE HIGH FLUX TRIP NOT BEING RESET.
(8026 DEFICIENCY)

10 CFR 50, APPENDIX A, CRITERION 23, STATES THAT THE PROTECTION SYSTEM SHALL BE DESIGNED TO FAIL INTO A SAFE STATE OR INTO A STATE DEMONSTRATED TO BE ACCEPTABLE ON SOME OTHER DEFINED BASIS IF CONDITIONS SUCH AS DISCONNECTION OF THE SYSTEM, LOSS OF ENERGY (E.G., ELECTRIC POWER, INSTRUMENT AIR), OR POSTULATED ADVERSE ENVIRONMENTS (E.G., EXTREME HEAT OR COLD, FIRE, PRESSURE, STEAM, WATER AND RADIATION) ARE EXPERIENCED. IN THE FASE APPENDIX 1A, PAGE 1A-26, AS PART OF A DISCUSSION ON CRITERION 23, THE LICENSEE STATES, "SAFETY FEATURES EQUIPMENT CAN BE MANUALLY INITIATED BY THE OPERATOR AT ANY TIME EVEN IF POWER IS LOST TO THE ACTUATION SYSTEM." CONTRARY TO THE REQUIREMENTS, THE REACTOR BUILDING PURGE INLET LINE VALVE, SFV 53503, THE REACTOR BUILDING PURGE OUTLET LINE VALVE SFV 53604, AND THE REACTOR BUILDING PRESSURE EQUALIZING LINE VALVE SFV 53210 WILL APPARENTLY FAIL OPEN ON LOSS OF DIRECT CURRENT POWER TO THE SOLENOID OPERATORS.
(8026 DEVIATION)

INSPECTION STATUS

SYSTEMS AND COMPONENT PROBLEMS:

- NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

- AWAITING DECISION OF ASLB. REGION V FOLLOWUP FROM OCTOBER 28 TO NOVEMBER 6, 1980 ON PERFORMANCE APPRAISAL BRANCH INSPECTION FINDINGS.

PLANT STATUS:

- OPERATION AT OR NEAR FULL POWER FOR MOST OF PERIOD.

LAST IE SITE INSPECTION DATE: 10/28-11/06/80+

INSPECTION REPORT NO. 50-312/80-34+

NRC POWER RESTRICTIONS: None

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-42/04T-0	10-09-80	10-28-80	SIX OF 19 ENVIRONMENTAL TLD'S INDICATE REPORTABLE EXPOSURES IN THIRD QUARTER. (SPECIAL REPORT)
80-43/01T-0	10-31-80	11-17-80	ASCO SOLENOID VALVES ON AIR-OPERATED CONTROL ISOLATION VALVES NOT ENVIR QUALIFIED. (SPECIAL REPORT)
80-44/01T-0	10-31-80	11-17-80	MELETRON PS ON MAIN STEAM LINES NOT ENVIR QUALIFIED.
80-45/(Prelim)	10-31-80	11-03-80	NAMCO POSITION SWITCHES ON CONTAINMENT ISOLATION VALVES NOT ENVIR QUALIFIED. (SPECIAL REPORT)

SUMMARY

INSPECTION SUMMARY

- INSPECTION ON SEPTEMBER 29 - OCTOBER 3, 1980 (REPORT NO. 50-312/80-27)
  AREAS INSPECTED: ROUTINE, UNANNOUNCED INSPECTION OF THE DESIGN CONTROL PROGRAM; THE NONLICENSED PERSONNEL TRAINING PROGRAM; FOLLOWUP ON PREVIOUS INSPECTION FINDINGS; FOLLOWUP ON IE BULLETINS/CIRCULARS; AND INDEPENDENT INSPECTION EFFORT. THE INSPECTION INVOLVED 34 INSPECTOR HOURS ONSITE BY ONE NRC INSPECTOR.

  RESULTS: NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

- INSPECTION ON SEPTEMBER 1-30, 1980 (REPORT NO. 50-312-80/29)
  AREAS INSPECTED: ROUTINE, OPERATIONAL SAFETY VERIFICATION; MONTHLY MAINTENANCE OBSERVATION; MONTHLY SURVEILLANCE OBSERVATION; REVIEW OF PLANT OPERATIONS; FOLLOWUP ON REGIONAL REQUESTS; IN-OFFICE REVIEW OF LICENSEE EVENT REPORTS; FOLLOWUP ON INSPECTOR IDENTIFIED PROBLEMS; AND INDEPENDENT INSPECTION EFFORT.

  RESULTS: NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

- INSPECTION ON OCTOBER 20-23, 1980 (REPORT NO. 50-312/80-30)
  AREAS INSPECTED: INCLUDED A REVIEW OF: SECURITY PROGRAM AUDIT; SECURITY ORGANIZATION; PHYSICAL BARRIER (PROTECTED AREA); LIGHTING; ACCESS CONTROL (ESCORTING); ACCESS CONTROL (VITAL AREAS); ASSESSMENT AIDS; TESTING AND MAINTENANCE; COMPENSATORY MEASURES; AND RECORDS AND REPORTS. THE INSPECTION INVOLVED 56 INSPECTOR HOURS ONSITE BY TWO NRC INSPECTORS.

  RESULTS: NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

- INSPECTION ON NOVEMBER 12-19, 1980 (REPORT NO. 50-312/80-31)
  INSPECTION BEING CONDUCTED; TO BE REPORTED NEXT MONTH.

- INSPECTION ON SEPTEMBER 22 - OCTOBER 3, 1980 (REPORT NO. 50-312/80-32)
  REPORT BEING PREPARED; TO BE REPORTED NEXT MONTH.

- INSPECTION ON OCTOBER 1-31, 1980 (REPORT NO. 50-312/80-33)
  REPORT BEING PREPARED; TO BE REPORTED NEXT MONTH.

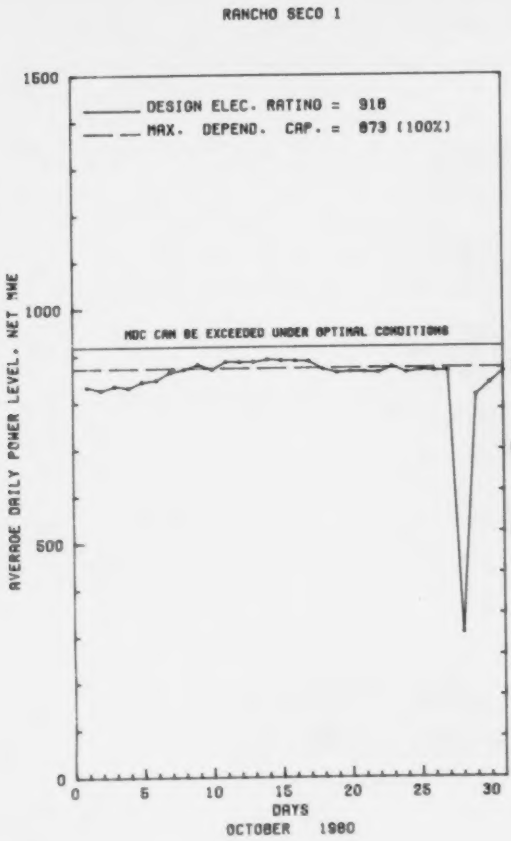
- INSPECTION ON OCTOBER 28 - NOVEMBER 6, 1980 (REPORT NO. 50-312/80-34)
  REPORT BEING PREPARED; TO BE REPORTED NEXT MONTH.

REVIEWED BY OMPA LOEB: RE Bobe **DATE:** 12/10/80

REVIEWED BY NRR DL: **DATE:** 12/10/80

RANCHO SECO

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-312
2. Utility Contact: R. HOLLINGSWORTH (916) 452-3211
3. Licensed Thermal Power (MWt): 2772
4. Nameplate Rating (Gross MWe): 1070 x 0.9 = 963
5. Design Electrical Rating (Net MWe): 918
6. Maximum Dependable Capacity (Gross MWe): 917
7. Maximum Dependable Capacity (Net MWe): 873
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	745.0	7,320.0	48,577.0
12. Hours Reactor Was Critical	742.3	3,958.7	30,032.2
13. Reactor Reserve Shutdown Hours	.0	494.5	4,469.6
14. Hours Generator On-Line	734.4	3,840.3	28,781.6
15. Unit Reserve Shutdown Hours	.0	.0	1,210.2
16. Gross Thermal Energy (MWH)	1,945,244	9,965,408	72,999,106
17. Gross Electrical Energy (MWH)	653,405	3,320,891	24,605,427
18. Net Electrical Energy (MWH)	620,678	3,134,890	23,245,780
19. Unit Service Factor	98.6	52.5	59.2
20. Unit Availability Factor	98.6	52.5	61.7
21. Unit Capacity Factor (MDC Net)	95.4	49.1	54.8
22. Unit Capacity Factor (DER Net)	90.8	46.7	52.1
23. Unit Forced Outage Rate	1.4	13.9	30.1

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
REFUELING OUTAGE IN MAY, 1981 - APPROX. 45 DAYS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
8	10/28/80	F	10.6	A	3		PA	AIRDRY	LOSS OF INSTRUMENT AIR PRESSURE RESULTED IN FEEDWATER FLOW IMBALANCE WHICH, IN TURN, LED TO REACTOR TRIP.

SUMMARY

RANCHO SECO 1 OPERATED AT OR NEAR FULL POWER FOR OCTOBER WITH ONE OUTAGE ON THE 28TH FOR EQUIPMENT FAILURE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

ROBINSON 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1. Location (state/county) (nearest population center)	South Carolina/Darlington 5miles NW of Hartsville
2. Electric Reliability Council	Southeastern Electric Reliability Council
3. Type of Reactor	PWR
4. Power Level (MWT)	2300
5. Design Electrical Rating (MWE Net)	700
6. Date of Initial Criticality	September 20, 1970
7. Date of Commercial Operation	March 7, 1971
8. Condenser Cooling Method	Recirculation from impounded lake
9. Condenser Cooling Water Source	Robinson Impoundment

Utility & Contractor Information

10. Licensee	Carolina Power and Light Company
11. Corporate Address	411 Fayetteville Street Raleigh, North Carolina
12. Corporate Contact	J. A. Jones Executive Vice President
13. Architect/Engineer	Ebasco
14. Nuclear Steam Supply System	Westinghouse
15. Constructor	Ebasco

Regulatory Information

16. IE Region responsible	II
17. IE Branch Chief	R.C. Lewis
18. IE Section Chief	P. J. Kellogg
19. IE Resident Inspector	A. K. Hardin
20. Licensing Project Manager	D. Neighbors
21. Docket Number	50-261
22. License Number/Date of Issue	DPR-23, 7/31/70 (5 Mwt), Full power authorization issued 9/23/70
23. Public Document Room	Hartsville Memorial Library Home and Fifth Avenue Hartsville, South Carolina 29550

INSPECTION STATUS

ENFORCEMENT STATUS

AS REQUIRED BY SECTION 6.8.1 OF THE TECHNICAL SPECIFICATIONS "WRITTEN PROCEDURES...SHALL BE ESTABLISHED, IMPLEMENTED, AND MAINTAINED THAT MEET...THE REQUIREMENTS...OF SECTION 5.1 AND 5.3 BY ANSI N18.7-1976 AND APPENDIX A". CONTRARY TO THE ABOVE, AS OF SEPTEMBER 8, 1980, PROCEDURE FOR PIPE STRESS REVIEW AND ANALYSIS HAD NO REFERENCE OF ACCEPTANCE CRITERIA ON DIMENSIONAL TOLERANCES FOR SAFETY RELATED SUPPORTS. (8024 INFRACTION)

AS REQUIRED BY SECTION 6.8.1 OF THE TECHNICAL SPECIFICATIONS "WRITTEN PROCEDURES...SHALL BE ESTABLISHED, IMPLEMENTED, AND MAINTAINED..." AND SPECIAL PROCEDURE SP-205 PARAGRAPH 6.28 "CLEARANCES SHALL BE MEASURED AND RECORDED ON ALL FOUR SIDES OF PIPE AND PENETRATION AND RESTRAINTS. CONTRARY TO THE ABOVE, FIELD DOCUMENT, USED TO VERIFY THE AS-BUILT FOR SAFETY-RELATED SYSTEM ISO-RC-4 DID NOT HAVE ANY RECORDS OF INSPECTION OF CLEARANCES FOR THE WALL PENETRATIONS. (8024 INFRACTION)

THE FIRE PREVENTION WELDING PROCEDURES WERE NOT BEING IMPLEMENTED AS REQUIRED BY TECHNICAL SPECIFICATION SECTION 6.8. (8026 INFRACTION)

THE FIRE PROTECTION FIRE PUMP, CONTROLLER AND PIPING BEING INSTALLED WERE NOT PROPERLY STORED IN ACCORDANCE WITH THE PLANT PROCEDURES AS REQUIRED BY TECHNICAL SPECIFICATION SECTION 6.8. (8026 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

STARTUP FROM REFUELING

♦ LAST IE SITE INSPECTION DATE: OCTOBER 2, 1980

INSPECTION REPORT NO: 50-261/80-26

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-019/03L-0	08/22/80	09/19/80	CANOPY SEAL LEAKS AT CONTROL RODS D4, L5 AND K6.
80-021/01T-0	09/18/80	10/02/80	CONTAINMENT SPRAY PIPING INSIDE OVER STRESSED ON DBE.
80-022/01T-0	09/18/80	10/02/80	B AND B STM GENERATOR EDDY CURRENT RESULTS MORE THAN 1 PERCENT DEFECTIVE.

SUMMARY

INSPECTION SUMMARY

♦ INSPECTION SEPTEMBER 8-11 (80-24): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 27 INSPECTOR-HOURS ONSITE IN THE AREAS OF SEISMIC ANALYSIS FOR AS-BUILT SAFETY-RELATED PIPING SYSTEMS (IE BULLETIN 79-14) AND PIPE SUPPORT BASE PLATE DESIGN USING CONCRETE EXPANSION ANCHOR BOLTS (IE BULLETIN 79-02). OF THE TWO AREAS INSPECTED, TWO ITEMS OF NONCOMPLIANCE WERE FOUND IN ONE AREA (INFRACTION - FAILURE TO ESTABLISH ADEQUATE PROCEDURES; INFRACTION - FAILURE TO FOLLOW PROCEDURES).

INSPECTION SEPTEMBER 22-26 (80-25): THIS ROUTINE UNANNOUNCED INSPECTION INVOLVED 33 INSPECTOR-HOURS ON SITE IN THE AREAS OF LICENSEE ACTION ON PREVIOUSLY IDENTIFIED ITEMS, QA PROGRAM ANNUAL REVIEW, ORGANIZATION AND ADMINISTRATION, PERSONNEL QUALIFICATION PROGRAM, LICENSEE ACTION ON PREVIOUS INSPECTOR IDENTIFIED ITEMS, PLANT TOUR AND GENERAL OPERATIONS. OF THE SEVEN AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION SEPTEMBER 29 - OCTOBER 2 (80-26): THIS SPECIAL, UNANNOUNCED INSPECTION INVOLVED 26 INSPECTOR-HOURS ON SITE IN THE AREAS OF FIRE PROTECTION/PREVENTION. OF THE AREAS INSPECTED, TWO ITEMS OF NONCOMPLIANCE WERE FOUND (INFRACTION - INADEQUATE STORAGE OF FIRE PROTECTION EQUIPMENT AND COMPONENTS; AND, INFRACTION - FAILURE TO FOLLOW REQUIREMENTS OF FIRE PREVENTION WELDING PROCEDURES); NO APPARENT DEVIATIONS WERE FOUND.

INSPECTION SEPTEMBER 4 (80-27): THIS SPECIAL, ANNOUNCED INSPECTION INVOLVED 8 INSPECTOR-HOURS AT THE HARRIS ENERGY AND ENVIRONMENTAL CENTER REVIEWING THE CIRCUMSTANCES SURROUNDING RECENT ERRONEOUSLY RECORDED PERSONNEL EXPOSURES FOR THE BRUNSWICK AND H. B. ROBINSON FACILITIES. OF THE ONE AREA INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

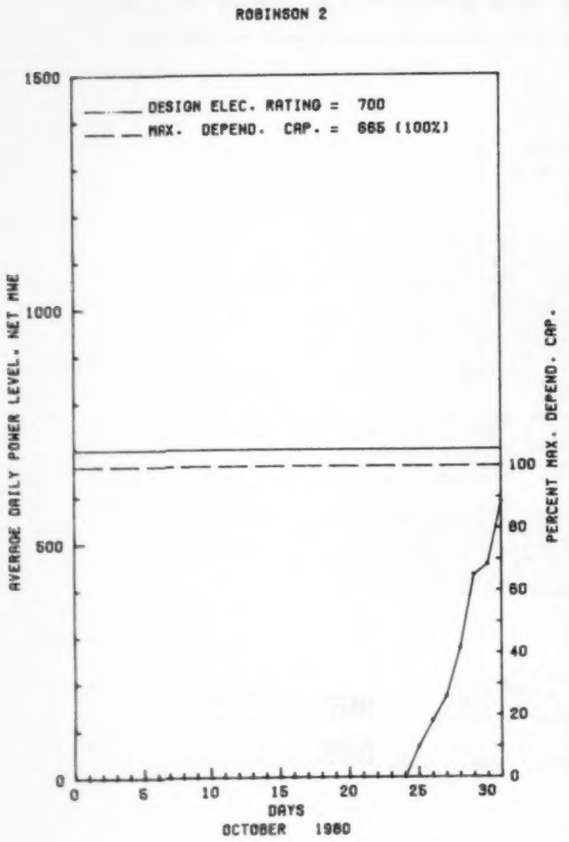
INSPECTION SEPTEMBER 8-12 (80-28): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 34 INSPECTOR-HOURS ON SITE IN THE AREAS OF PROCEDURE REVIEW, LICENSEE EVENT REPORTS AND OPERATIONS. OF THE THREE AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

REVIEWED BY OMPA LOEB: PE Bobe **DATE:** 12/10/80

REVIEWED BY NRR DL: Don Neighbors **DATE:** 12/11/80

ROBINSON 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-261
2. Utility Contact: M. L. Watford (803) 383-4524
3. Licensed Thermal Power (MWt): 2300
4. Nameplate Rating (Gross MWe): 854 x 0.9 = 769
5. Design Electrical Rating (Net MWe): 700
6. Maximum Dependable Capacity (Gross MWe): 700
7. Maximum Dependable Capacity (Net MWe): 665
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	745.0	7,320.0	84,702.0
12. Hours Reactor Was Critical	192.2	4,498.9	64,705.8
13. Reactor Reserve Shutdown Hours	3.3	223.1	942.4
14. Hours Generator On-Line	151.8	4,434.4	63,125.3
15. Unit Reserve Shutdown Hours	.0	.0	23.2
16. Gross Thermal Energy (MWH)	177,965	8,385,818	128,936,147
17. Gross Electrical Energy (MWH)	54,576	2,659,591	41,596,104
18. Net Electrical Energy (MWH)	49,858	2,503,955	39,409,057
19. Unit Service Factor	20.4	60.6	74.5
20. Unit Availability Factor	20.4	60.6	74.6
21. Unit Capacity Factor (MDC Net)	10.1	51.4	70.0
22. Unit Capacity Factor (DER Net)	9.6	48.9	66.5
23. Unit Forced Outage Rate	5.5	18.9	13.6

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
08-01	08/08/80	S	576.8	C	4		RC	FUELXX	CONTINUATION OF MAINTENANCE/REFUELING OUTAGE REPORTED FOR AUGUST AND SEPTEMBER.
10-01	10/25/80	S	2.3	B	1		HA	TURBIN	SHUTDOWN TO CHECK OVERSPEED TRIP.
10-02	10/25/80	F	3.5	B	1		HA	GENERA	GENERATOR GROUNDING STRAP RECONNECTED.
10-03	10/25/80	F	7.2	A	3		CH	VALVEX	STEAM FLOW/FEEDWATER FLOW MISMATCH AND 30% STEAM GENERATOR LEVEL.
10-04	10/26/80	F	3.4	A	3		CH	VALVEX	"B" STEAM GENERATOR HI-HI LEVEL - "B" FEEDWATER REGULATOR VALVE SWUNG OPEN. VALVE WAS STROKED AND LUBRICATED.

SUMMARY

ROBINSON 2 RETURNED ON-LINE ON THE 25TH OF OCTOBER FROM REFUELING AND ENCOUNTERED FOUR OUTAGES ON THE 25TH AND 26TH FOR MAINTENANCE AND EQUIPMENT FAILURE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

SALEM 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	New Jersey/Salem 20 miles S of Wilmington, Del.
2.	Electric Reliability Council	Mid-Atlantic Area Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	3338
5.	Design Electrical Rating (MWE Net)	1090
6.	Date of Initial Criticality	December 11, 1976
7.	Date of Commercial Operation	June 30, 1977
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Delaware River

Utility & Contractor Information

10.	Licensee	Public Service Electric & Gas Co.
11.	Corporate Address	80 Park Place - 15A Newark, New Jersey 07101
12.	Corporate Contact	F. P. Librizzi General Manager - Electric Prod.
13.	Architect/Engineer	PSE&G
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	UE&C

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	IE Section Chief	R. R. Keimig
19.	IE Resident Inspector	L. J. Norrholm (609-935-3850)
20.	Licensing Project Manager	W. Ross
21.	Docket Number	50-272
22.	License Number/Date of Issue	DPR-70, 8/13/76
23.	Public Document Room	Salem Free Public Library 112 West Broadway Salem, New Jersey 08079

INSPECTION STATUS

ENFORCEMENT STATUS

FAILURE TO ADOPT AND IMPLEMENT A REQUIREMENT OF AN ORDER MODIFYING LICENSE EFFECTIVE IMMEDIATELY ISSUED ON MARCH 28, 1980. THE ORDER REQUIRED CONTINUOUS VISUAL SURVEILLANCE OF ALL INTERFACE DOORS LEADING TO VITAL AREAS IN UNIT 1.
(8012 VIOLATION)

T.S. 6.8.1 REQUIRES WRITTEN PROCEDURES TO BE IMPLEMENTED COVERING THE ACTIVITIES OF REG. GUIDE 1.33 APP. A. AP-22 REQUIRES THAT CALIBRATION AND TEST EQUIPMENT BE CHECKED IN ACCORDANCE WITH THE VENDOR'S MANUAL REQUIREMENTS. THE VENDOR'S MANUAL REQUIRES SPECIFIC ENVIRONMENTAL CONDITIONS DURING CALIBRATION FOR JOHN FLUKE DVMS. CONTRARY TO THE ABOVE THE LICENSEE WAS UNABLE TO VERIFY THAT THESE CONDITIONS HAD BEEN MET DURING THE CALIBRATION OF JOHN FLUKE DVM PD-003, 004 AND 187. ALL WERE SUBSEQUENTLY USED TO CALIBRATE SAFETY RELATED EQUIPMENT.
(8019 DEFICIENCY)

10 CFR, APP. B, CRITERION V REQUIRES ACTIVITIES AFFECTING QUALITY TO BE PRESCRIBED BY PROCEDURES AND ACCOMPLISHED BY THEM. THE FOLLOWING IS A LIST OF THE LICENSEE'S FAILURE TO FOLLOW PROCEDURES: AP-19 REQUIRES THAT REJECTED RECEIVED ITEMS BE TAGGED AND STORED IN A HOLD AREA. CONTRARY TO THE REQUIREMENT LOT 26, BORIC ACID SHIPMENT P.O. 727471 WAS NOT HELD IN A QA HOLD AREA OR TAGGED PROPERLY. 10 CFR APP B, CRITERION XVII REQUIRES INSPECTOR IDENTIFICATION ON RECORDS. CONTRARY TO THIS NONE OF THE WAREHOUSE INSPECTION RECORDS IDENTIFIED THE INSPECTOR. SR-8 REQUIRES MONTHLY INSPECTIONS OF SAFETY RELATED STORAGE AREAS AND THE COMPLETED INSPECTION REPORTS BE SENT TO THE TDR FOR MICROFILMING. CONTRARY TO THIS THE APRIL 80 REPORT WAS MISSING AND NONE OF THE INSPECTION RECORDS WERE BEING MICROFILMED.
(8019 DEFICIENCY)

DURING THIS INSPECTION CONDUCTED ON OCTOBER 6-7 AND 9-10, 1980 THE FOLLOWING APPARENT ITEM OF NONCOMPLIANCE WAS IDENTIFIED. THIS ITEM IS A DEFICIENCY. THE SALEM UNIT 1 TECHNICAL SPECIFICATION 6.8.1 STATES, IN PART, THAT "WRITTEN PROCEDURES SHALL BE ESTABLISHED...COVERING THE ACTIVITIES REFERENCED BELOW: A. THE APPLICABLE PROCEDURES RECOMMENDED IN APPENDIX "A" OF REGULATORY GUIDE 1.33, NOVEMBER 1972..." REGULATORY GUIDE 1.33, NOVEMBER 1972, APPENDIX "A", SECTION I.1 STATES, IN PART, THAT "MAINTENANCE WHICH CAN AFFECT THE PERFORMANCE OF SAFETY-RELATED EQUIPMENT SHOULD BE...PERFORMED IN ACCORDANCE WITH WRITTEN PROCEDURES,..." MAINTENANCE PROCEDURE M11F, PARAGRAPH 3.1, STATES, IN PART, THAT "...WHEN THE REACTOR VESSEL IS OPEN, THE CONTROL AREA IS THE REACTOR VESSEL PIT..." THIS PROCEDURE FURTHER STATES, IN PART, THAT "CONTROL OF TOOLS AND ITEMS IN AN AREA IS BY SECURING ESSENTIAL PERSONAL ITEMS...BY TAPING TO THE BODY..." CONTRARY TO THE ABOVE, ON OCTOBER 10, 1980, MEMBERS OF A WORK CREW, INCLUDING A SUPERVISOR WORKING FROM A POSITION OVER THE REACTOR VESSEL PIT, DID NOT HAVE ESSENTIAL PERSONAL ITEMS TAPED TO THEIR BODY OR OTHERWISE FIRMLY SECURED.
(8024 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

- SYSTEM AND COMPONENT PROBLEMS:
- NONDESTRUCTIVE TESTING (NDT) REVEALED THAT #13 LOW PRESSURE TURBINE HAD A CRACK IN THE THIRD STAGE ROTOR. THE ROTOR WAS REMOVED AND A COLLAR INSTALLED TO MAINTAIN SPACING BETWEEN THE ADJACENT ROTORS. A NEW ROTOR WILL BE INSTALLED AT THE NEXT REFUELING OUTAGE.
- DURING REFUELING, MINOR GRID STRAP DAMAGE WAS OBSERVED ON ONE ASSEMBLY WHICH HAD NOT PREVIOUSLY BEEN IDENTIFIED. THIS ASSEMBLY HAD BEEN SELECTED FOR MONITORING BECAUSE THE ADJACENT ASSEMBLY HAD GRID STRAP DAMAGE OBSERVED DURING THE LAST REFUELING. AN EVALUATION IS IN PROGRESS.
- FACILITY ITEMS (PLANS AND PROCEDURES):
- ANTICIPATED COMPLETION OF REFUELING OUTAGE MID-DECEMBER 1980.

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-047/ 03L	08/29/80	09/26/80	OPENED FIRE BARRIER WITHOUT ESTABLISHING FIRE WATCH
80-051/ 03L	09/23/80	10/21/80	FAN BELT FAILURE ON EXHAUST FANS 12 AND 13 FOR FUEL HANDLING BUILDING
80-052/ 03L	09/10/80	10/10/80	MISSED SURVEILLANCE - INSERVICE INSPECTION OF VALVES PER TS 405

PLANT STATUS:

- PLANT IS IN A REFUELING OUTAGE.
- LAST IE SITE INSPECTION DATE: 10/14-17/80
- INSPECTION REPORT NO: 50-272/80-28

SUMMARY

INSPECTION SUMMARY

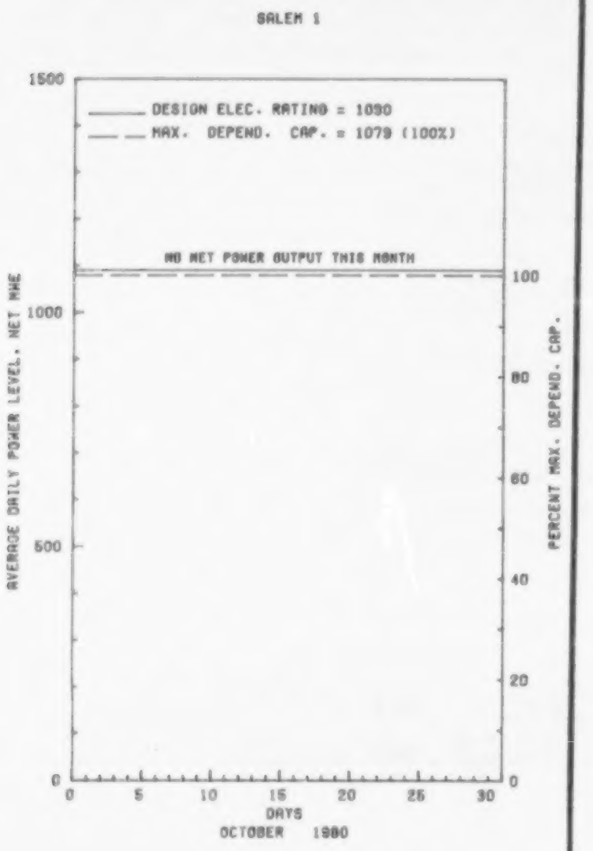
- 50-272/80-12 - APR 18: ENFORCEMENT MANAGEMENT MEETING WAS HELD WITH THE LICENSEE PURSUANT TO AN ORDER MODIFYING LICENSE EFFECTIVE IMMEDIATELY TO DISCUSS, INTER ALIA, THE ADOPTION AND IMPLEMENTATION OF MODIFICATIONS TO LICENSE ISSUED AS PART OF AN ENFORCEMENT LETTER DATED MARCH 28, 1980 AND PROPOSED CORRECTIVE ACTION TO THE ITEMS OF NONCOMPLIANCE. ALSO DISCUSSED WAS THE AUXILIARY FEEDWATER PUMP DISCHARGE VALVE PROBLEM. THE PROCEDURE ADOPTED AND IMPLEMENTED IN RESPONSE TO THE MODIFICATION IN PART III-A OF THE ORDER DID NOT MEET THE REQUIREMENT OF THE ORDER; THEREFORE, FROM APPROXIMATELY MARCH 28, 1980, DATE OF RECEIPT OF THE ORDER, UNTIL APRIL 14, 1980, AT WHICH TIME THE NEED FOR THE PROVISIONS OF THE ORDER WERE ELIMINATED, THE LICENSEE IS DEEMED TO HAVE BEEN IN APPARENT NONCOMPLIANCE WITH THE ORDER.
- 50-272/80-21 - SEP 1 - OCT 11: ROUTINE INSPECTION BY TWO RESIDENT INSPECTORS (60 HRS) OF PLANT OPERATIONS INCLUDING TOURS OF THE FACILITY; LOG AND RECORD REVIEWS; REVIEW OF LICENSEE EVENTS; OPERATIONAL SAFETY VERIFICATION; IE BULLETINS; PREPARATIONS FOR REFUELING; LICENSEE EVENT REPORTS; AND, FOLLOWUP ON PREVIOUS INSPECTION ITEMS. NO NONCOMPLIANCES WERE IDENTIFIED.
- 50-272/80-24 - OCT 6-7 & 9-10: ROUTINE, UNANNOUNCED INSPECTION BY ONE REGION-BASED INSPECTOR (34 HRS) OF LICENSEE ACTION ON PREVIOUS INSPECTION FINDINGS AND INSERVICE INSPECTION (ISI) ACTIVITIES INCLUDING REVIEW OF NDE PROCEDURES, REVIEW OF DATA, PROGRAM PLAN REVIEW, OBSERVATIONS OF NDE IN PROGRESS, ACTIVITIES ASSOCIATED WITH USE OF MECHANIZED PAR INSPECTION DEVICE AND LICENSEE AUDITS OF THE ISI CONTRACTOR. ONE NONCOMPLIANCE WAS IDENTIFIED: FAILURE TO CONTROL ESSENTIAL PERSONAL ITEMS.

REVIEWED BY OMPA LOEB: ______ **DATE:** 12/10/80

REVIEWED BY NRR DL: ______ **DATE:** 12/11/80
for W. Ross

SALEM 1

AVERAGE DAILY POWER LEVEL(MWe)



OPERATING STATUS

1. Docket Number: 50-272
2. Utility Contact: L. K. Miller (609) 365-7000
3. Licensed Thermal Power (MWt): 3338
4. Nameplate Rating (Gross MWe): 1300 x 0.9 = 1170
5. Design Electrical Rating (Net MWe): 1090
6. Maximum Dependable Capacity (Gross MWe): 1124
7. Maximum Dependable Capacity (Net MWe): 1079
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	745.0	7,320.0	29,281.0
12. Hours Reactor Was Critical	.0	6,097.8	16,184.1
13. Reactor Reserve Shutdown Hours	.0	.0	22.7
14. Hours Generator On-Line	.0	5,966.3	15,477.4
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	18,205,037	45,741,333
17. Gross Electrical Energy (MWH)	0	5,935,090	15,102,590
18. Net Electrical Energy (MWH)	-2,933	5,661,891	14,285,431
19. Unit Service Factor	.0	81.5	52.9
20. Unit Availability Factor	.0	81.5	52.9
21. Unit Capacity Factor (MDC Net)	.0	71.7	45.2
22. Unit Capacity Factor (DER Net)	.0	71.0	44.8
23. Unit Forced Outage Rate	.0	5.5	34.4

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 12/07/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-210	09/19/80	S	745.0	C	4		RC	FUELXX	ANNUAL REFUELING OUTAGE CONTINUES.

SUMMARY

SALEM 1 REMAINED SHUTDOWN DURING OCTOBER FOR REFUELING.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

SAN ONOFRE 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	California/San Diego 5 miles S of San Clemente
2.	Electric Reliability Council	Western Systems Coordinating Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	1347
5.	Design Electrical Rating (MWE Net)	436
6.	Date of Initial Criticality	June 14, 1967
7.	Date of Commercial Operation	January 1, 1968
8.	Condenser Cooling Method	Circulating sea water
9.	Condenser Cooling Water Source	Pacific Ocean

Utility & Contractor Information

10.	Licensee	Southern California Edison Co.
11.	Corporate Address	2244 Walnut Grove Avenue Rosemead, California 91770
12.	Corporate Contact	R. E. Woodbury V.P. & General Counsel
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	V
17.	IE Branch Chief	J. L. Crews
18.	IE Section Chief	B. H. Faulkenberry
19.	IE Resident Inspector	L. F. MILLER (714-492-2641)
20.	Licensing Project Manager	S. J. Nowicki
21.	Docket Number	50-206
22.	License Number/Date of Issue	DPR-13, 3/27/67
23.	Public Document Room	Mission Viejo Branch Library 24851 Chrisanta Drive Mission Viejo, California 92676

INSPECTION STATUS

ENFORCEMENT STATUS

TECHNICAL SPECIFICATION 6.8.1 REQUIRES THAT WRITTEN PROCEDURES AND ADMINISTRATIVE POLICIES SHALL BE ESTABLISHED, IMPLEMENTED AND MAINTAINED THAT MEET OR EXCEED THE REQUIREMENTS AND RECOMMENDATIONS OF APPENDIX A OF USNRC REGULATORY GUIDE 1.33, REV. 1. PARAGRAPH 10 OF THIS APPENDIX STATES THAT PROCEDURES SHOULD BE WRITTEN TO PRESCRIBE THE INSTRUCTIONS FOR MAINTAINING WATER QUALITY WITHIN PRESCRIBED LIMITS, AND THE LIMITS ON CONCENTRATIONS OF AGENTS THAT MAY CAUSE CORROSIVE AFFECT. STATION ORDER S-E-2, "OPERATION, MAINTENANCE AND CHEMICAL CONTROL OF HEAT EXCHANGER EQUIPMENT," PROVIDES THAT "CORROSION OF THE PRIMARY SYSTEM COMPONENTS WILL BE CONTROLLED BY MAINTAINING THE CHLORIDE LEVELS WITHIN THE DEFINED LIMITS." PARAGRAPH 8.1.B DEFINES THE CHLORIDE LIMIT AS "LESS THAN 0.15 PPM. CONTRARY TO THE REQUIREMENTS, ON EACH DAILY SAMPLE RECORDED FROM SEPTEMBER 2 TO SEPTEMBER 13, 1980, THE CHLORIDE CONCENTRATION WAS IN EXCESS OF THE SPECIFIED LIMIT BY AN AMOUNT WHICH VARIED FROM 0.03 TO 0.35 PPM, OR 20-233% IN EXCESS OF THE LIMIT.
(8028 INFRACTION)

TECHNICAL SPECIFICATION 6.8.1 REQUIRES THAT WRITTEN PROCEDURES AND ADMINISTRATIVE POLICIES SHALL BE ESTABLISHED, IMPLEMENTED AND MAINTAINED THAT MEET OR EXCEED THE REQUIREMENTS AND RECOMMENDATIONS OF APPENDIX A OF USNRC REGULATORY GUIDE 1.33, REV. 1. PARAGRAPH 7A(3), "LIQUID RADIOACTIVE WASTE SYSTEM: DISCHARGING THE EFFLUENTS," IS A RECOMMENDED PROCEDURE OF THIS APPENDIX. THE LICENSEE'S OPERATING INSTRUCTION S-3-2.26, "RECEIVING, STORAGE, PROCESSING AND DISCHARGE OF LIQUID WASTE," IMPLEMENTS THE APPENDIX A RECOMMENDATION. PRECAUTION III A OF THIS INSTRUCTION STATES THAT "DURING HOLDUP TANK RELEASES, THE SET POINT FOR ORMS 1218 SHALL BE SET AT A MAXIMUM OF 20,000 CPM ABOVE BACKGROUND." CONTRARY TO THE REQUIREMENTS, ON SEPTEMBER 24, 1980, WHILE THE WEST HOLDUP TANK WAS BEING DISCHARGED TO THE CIRCULATING WATER SYSTEM, THE INSPECTOR OBSERVED THAT THE SET POINT FOR URMS 1218 WAS SET AT 110,000 CPM, 95,000 CPM ABOVE THE BACKGROUND COUNT RATE OF APPROXIMATELY 15,000 CPM.
(8028 DEFICIENCY)

INSPECTION STATUS

SYSTEMS AND COMPONENT PROBLEMS:

- NONE.

LARGE NUMBER OF TUBES DETERMINED TO NEED PLUGGING (RESULTS OF EDDY CURRENT TESTING). LICENSEE PLANS TO INSTALL SLEEVES IN TUBES IN LIEU OF PLUGGING.

FACILITY ITEMS (PLANS AND PROCEDURES):

STEAM GENERATOR REPAIRS.

MANAGERIAL ITEMS

NONE.

PLANT STATUS:

PLANT SHUTDOWN FOR STEAM GENERATOR REPAIRS.

LAST IE SITE INSPECTION DATE: 11/03-07/80+

INSPECTION REPORT NO: 50-206/80-29+

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-36/03L-0	09-22-80	10-03-80	INADVERTENT DILUTION OF REACTOR COOLANT DURING SG DECONTAMINATION.
80-11-E1	10-27-80	11-03-80	LOSS OF THERMAL MEASUREMENT. (SPECIAL REPORT)

SUMMARY

INSPECTION SUMMARY

- INSPECTION ON SEPTEMBER 22-26, 1980 (REPORT NO. 50-206/80-26) REPORT BEING PREPARED; TO BE REPORTED NEXT MONTH.
- INSPECTION ON SEPTEMBER 15-19, 1980 (REPORT NO. 50-206/80-27) AREAS INSPECTED: ROUTINE, UNANNOUNCED INSPECTION OF ONSITE ORGANIZATION AND ADMINISTRATION; MAINTENANCE PROGRAM; NONLICENSED AND GENERAL TRAINING; REQUALIFICATION TRAINING; IE BULLETIN/CIRCULAR FOLLOWUP; FIRE PROTECTION MODIFICATIONS; AND INDEPENDENT INSPECTION EFFORT. THE INSPECTION INVOLVED 33 INSPECTOR HOURS ONSITE BY ONE NRC INSPECTOR.

  RESULTS: NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.
- INSPECTION ON SEPTEMBER 4-26, 1980 (REPORT NO. 50-206/80-28) AREAS INSPECTED: ROUTINE, RESIDENT INSPECTION OF PLANT OPERATIONS DURING LONG TERM OUTAGE; MONTHLY MAINTENANCE AND SURVEILLANCE OBSERVATIONS; AND INDEPENDENT INSPECTION EFFORT. THE INSPECTION INVOLVED 62 INSPECTOR HOURS ONSITE BY ONE NRC INSPECTOR.

  RESULTS: TWO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED (DEFICIENCY - FAILURE TO OBSERVE REACTOR COOLANT CHEMISTRY LIMIT AND DEFICIENCY - FAILURE TO FOLLOW RADIOACTIVE WASTE DISCHARGE PROCEDURE).
- INSPECTION ON NOVEMBER 3-7, 1980 (REPORT NO. 50-206/80-29) REPORT BEING PREPARED; TO BE REPORTED NEXT MONTH.
- INSPECTION ON OCTOBER 20-23, 1980 (REPORT NO. 50-206/80-30) AREAS INSPECTED: ROUTINE, UNANNOUNCED INSPECTION OF THE DESIGN CHANGES AND MODIFICATIONS PROGRAM; QA PROGRAM; CALIBRATION; IE BULLETIN/CIRCULAR FOLLOWUP; AND INDEPENDENT INSPECTION EFFORT. THE INSPECTION INVOLVED 26 INSPECTOR HOURS ONSITE BY ONE NRC INSPECTOR.

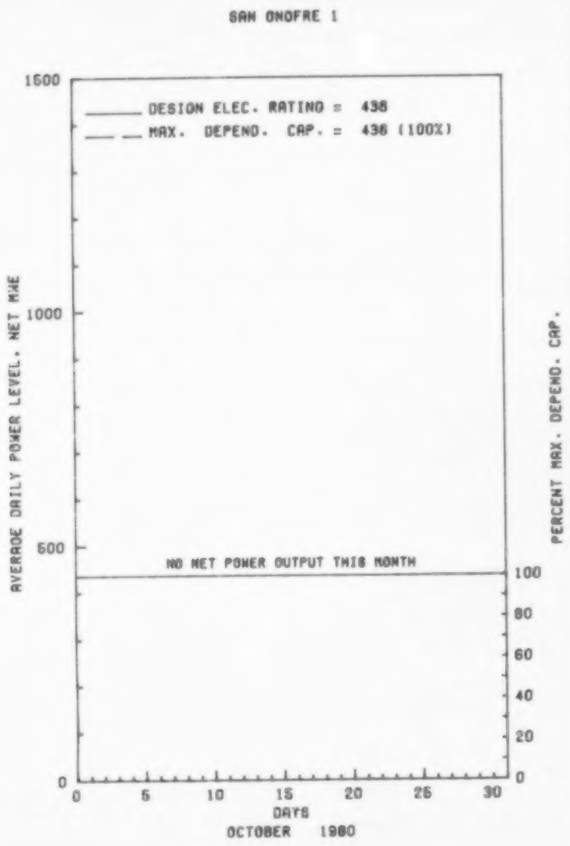
  RESULTS: NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.
- INSPECTION ON OCTOBER 1-31, 1980 (REPORT NO. 50-206/80-31) REPORT BEING PREPARED; TO BE REPORTED NEXT MONTH.

REVIEWED BY OMPA LOEB: BZ Robe **DATE:** 12/10/80

REVIEWED BY NRR DL: S.J. Nowicki **DATE:** 12/11/80

SAN ONOFRE 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-206
2. Utility Contact: J. M. JOY (714) 492-7700
3. Licensed Thermal Power (MWt): 1347
4. Nameplate Rating (Gross MWe): 500 x 0.9 = 450
5. Design Electrical Rating (Net MWe): 436
6. Maximum Dependable Capacity (Gross MWe): 456
7. Maximum Dependable Capacity (Net MWe): 436

CAPACITY FACTORS CALCULATED WITH A WEIGHTED AVERAGE

8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	745.0	7,320.0	117,296.0
12. Hours Reactor Was Critical	.0	1,979.1	84,611.3
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	.0	1,961.6	81,114.0
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	2,552,787	104,084,261
17. Gross Electrical Energy (MWH)	0	855,000	35,526,434
18. Net Electrical Energy (MWH)	0	816,676	33,664,042
19. Unit Service Factor	.0	26.8	69.2
20. Unit Availability Factor	.0	26.8	69.2
21. Unit Capacity Factor (MDC Net)	.0	25.6	70.2
22. Unit Capacity Factor (DER Net)	.0	25.6	70.2
23. Unit Forced Outage Rate	100.0	58.3	11.8

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 06/81

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
55	07/12/80	F	745.0	B	4	80-014	CC	HTEXCH	STEAM GENERATOR TUBE REPAIR CONTINUES.

SUMMARY

SAN ONOFRE 1 REMAINED SHUTDOWN DURING OCTOBER FOR STEAM GENERATOR TUBE REPAIR.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

SEQUOYAH 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1. Location (state/county) (nearest population center)	Tennessee/Hamilton 9.5 Miles NE of Chattanooga
2. Electric Reliability Council	SERC
3. Type of Reactor	PWR
4. Licensed Power Level (MWT)	3411
5. Design Electrical Rating (MWE Net)	1140
6. Date of Initial Criticality	July 5, 1980
7. Date of Commercial Operation	N/A
8. Condenser Cooling Method	Natural Draft Cooling Towers
9. Condenser Cooling Water Source	Tennessee River

Utility and Contractor Information

10. Licensee	Tennessee Valley Authority
11. Corporate Address	500A Chestnut Street Tower II Chattanooga, TN 37401
12. Corporate Contact	H. G. Parris, Manager of Power
13. Architect/Engineer	TVA
14. Nuclear Steam Supply System	Westinghouse
15. Constructor	TVA

Regulatory Information

16. IE Region responsible	II
17. IE Branch Chief	R. C. Lewis
18. IE Section Chief	H. C. Dance
19. IE Resident Inspector	S. D. BUTLER
20. Licensing Project Manager	C. R. Stahle
21. Docket Number	50-327
22. License Number/Date of Issue	DPR-77, 9/17/80
23. Public Document Room	Chattanooga - Hamilton Bicentennial Library 1001 Broad Street Chattanooga, Tennessee 37402

INSPECTION STATUS

ENFORCEMENT STATUS

AS REQUIRED BY APPENDIX A TECHNICAL SPECIFICATIONS, SECTION 3.6.3.1, CONTAINMENT ISOLATION VALVE FCV-61-192 SHALL BE OPERABLE WITH A MAXIMUM ISOLATION TIME OF LESS THAN OR EQUAL TO TEN (10) SECONDS WITH THE PLANT IN MODES 1, 2, 3, OR 4. CONTRARY TO THE ABOVE, ON JULY 27, 1980, CONTAINMENT ISOLATION VALVE FCV-61-192 WAS IDENTIFIED TO REQUIRE APPROXIMATELY 20 SECONDS TO STROKE CLOSED WITH THE PLANT IN MODE 3. NO ACTION WAS TAKEN TO COMPLY WITH THE LIMITING CONDITION FOR OPERATION FOR THE ACTION STATEMENT OF TECHNICAL SPECIFICATION 3.6.3.1.
(8036 INFRACTION)

AS REQUIRED BY APPENDIX A TECHNICAL SPECIFICATIONS, SECTION 6.8.1, WRITTEN PROCEDURES SHALL BE ESTABLISHED, IMPLEMENTED AND MAINTAINED COVERING THE APPLICABLE PROCEDURES IN APPENDIX A OF REGULATORY GUIDE 1.33, REVISION 2, FEBRUARY 1978. APPENDIX A OF REGULATORY GUIDE 1.33 REQUIRES THAT INSTRUCTION FOR STARTUP, SHUTDOWN, AND CHANGING MODES OF OPERATION BE PREPARED FOR THE EMERGENCY POWER SOURCES SYSTEMS. CONTRARY TO THE ABOVE, ON AUGUST 9, 1980, FAILURE TO FOLLOW THE SYSTEM OPERATING INSTRUCTIONS FOR RESETTING THE EMERGENCY DIESEL GENERATORS FOLLOWING AN AUTOMATIC START, RESULTED IN ALL FOUR EMERGENCY DIESEL GENERATORS BEING MADE INOPERABLE FOR APPROXIMATELY 24 MINUTES.
(8036 INFRACTION)

AS REQUIRED BY 10 CFR 73.55(D)(4), "...DESIGNATED LICENSEE VEHICLES SHALL BE LIMITED IN THEIR USE TO ONSITE PLANT FUNCTIONS...THE LICENSEE SHALL EXERCISE POSITIVE CONTROL OVER ALL DESIGNATED VEHICLES TO ASSURE THAT THEY ARE USED ONLY BY AUTHORIZED PERSONS AND FOR AUTHORIZED PURPOSES." IN ADDITION, SEQUOYAH NUCLEAR PLANT PHYSICAL SECURITY INSTRUCTION PHYSI-5, SECTION 5.2 STATES THAT "...ALL VEHICLES WITHIN THE PROTECTED AREA SHALL BE SECURED WHEN NOT IN USE. VEHICLES THAT CANNOT BE LOCKED BY KEY SHALL BE SECURED BY THE USE OF A LOCKING DEVICE...". CONTRARY TO THE ABOVE, ON AUGUST 6, 1980, THE INSPECTOR FOUND TWO DESIGNATED VEHICLES, NUMBERS 22 AND 33, UNLOCKED AND UNATTENDED IN THE PROTECTED AREAS.
(8036 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE

MANAGERIAL ITEMS:

NONE

PLANT STATUS:

- STARTUP TEST PROGRAM.
- LAST IE SITE INSPECTION DATE: OCTOBER 1 - NOVEMBER 3, 1980
- INSPECTION REPORT NO: 50-327/80-42

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-00[illegible]/ 03L-0	06/23/80	09/17/80	INADVERTENT SAFETY INJECTION DUE TO OPENING MAIN STEAM ISOLATION VALVES.
80-085/ 03L-0	08/04/80	09/25/80	WRONG KIND OF DETECTOR INSTALLED IN FIRE DETECTION SYSTEM.
80-088/ 03L-0	09/02/80	10/01/80	SAFETY RELATED CABLE FOUND NOT TO BE PROPERLY ENVIRONMENTALLY QUALIFIED.
80-129/ 03L-0	08/08/80	09/02/80	DIESEL 1A-A FAILED TO START DURING PREOP DUE TO SWITCH LEFT IN TEST.
80-138/ 03L-0	08/01/80	08/29/80	COOLING TOWER BLOWDOWN EFFLUENT MONITOR INOPERABLE.
80-141/ 01T-0	08/29/80	09/11/80	LEAK IN CONTROL ROD DRIVE HOUSING WELD.
80-142/ 03L-0	07/18/80	08/29/80	ICE BED TEMPERATURE EXCEEDED 27 DEGREES.
80-148/ 03L-0	09/04/80	10/02/80	SHIELD BUILDING VENT MONITOR INOPERABLE DUE TO BROKEN BELT ON SAMPLE PUMP.
80-151/ 03L-0	08/11/80	09/10/80	TURBINE AUXILIARY FEED PUMP FAILED TO START DUE TO FAILED LIMIT SWITCH ON STOP VALVE OPERATOR.

SUMMARY

INSPECTION SUMMARY

- INSPECTION AUGUST 11 AND 15 (80-33): VISIT TO CHEM-NUCLEAR WASTE DISPOSAL SITE, BARNWELL, SOUTH CAROLINA FOR UNANNOUNCED INSPECTIONS OF LICENSEE-SHIPPERS, AND AN AUGUST 25 VISIT TO SEQUOYAH NUCLEAR POWER STATION TO INSPECT RADIOACTIVE WASTE SHIPPING AND SOLIDIFICATION PROCEDURES. EACH LICENSEE-SHIPPER VEHICLE AT BARNWELL, SC WAS INSPECTED FOR COMPLIANCE WITH DEPARTMENT OF TRANSPORTATION (DOT) AND NUCLEAR REGULATORY COMMISSION (NRC) REGULATIONS AS FOLLOWS: (1) SHIPPING PAPER REQUIREMENTS; (2) DOT PLACARDING REQUIREMENTS; (3) RADIATION LEVELS; (4) REMOVABLE CONTAMINATION; (5) DOT MARKING AND LABELING REQUIREMENTS FOR PACKAGES; (6) DOT AND NRC REQUIREMENTS FOR EXTERNAL PACKAGE FEATURES, AND (7) PROHIBITED ARTICLES OR CONTENTS. THE INSPECTION AT SEQUOYAH NPS CONSISTS OF A REVIEW OF SHIPPING DOCUMENTS AND PROCEDURES FOR VENDOR SUPPLIED SOLIDIFICATION EQUIPMENT, FOLLOWING DISCOVERY OF LIQUID IN A RADIOACTIVE WASTE SHIPMENT AT BARNWELL, SC. OF THE SEVERAL AREAS INSPECTED NO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED.

INSPECTION AUGUST 1-31 (80-36): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 108 INSPECTOR-HOURS ON SITE IN THE AREAS OF OPERATIONAL SAFETY VERIFICATION, VERIFICATION OF LICENSE CONDITIONS, IE BULLETIN REVIEW, INDEPENDENT INSPECTION EFFORT AND FOLLOWUP ON PLANT INCIDENTS. OF THE FIVE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN TWO AREAS; THREE ITEMS OF NONCOMPLIANCE WERE FOUND IN THREE AREAS (INFRACTION - FAILURE TO CONTROL DESIGNATED VEHICLES; INFRACTION - FAILURE TO FOLLOW PROCEDURE; AND INFRACTION - FAILURE TO COMPLY WITH TECHNICAL SPECIFICATIONS).

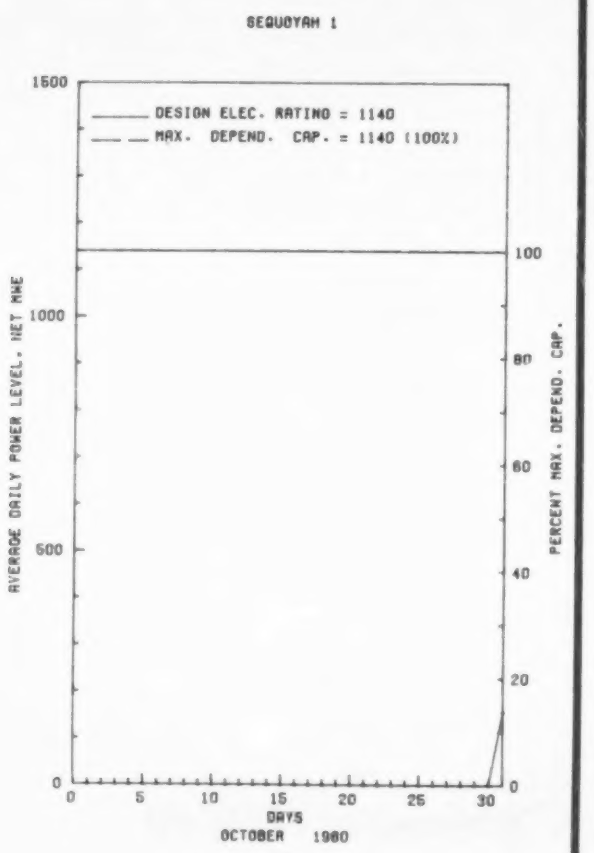
INSPECTION SEPTEMBER 16-19 (80-39): THIS ROUTINE UNANNOUNCED INSPECTION INVOLVED 28 INSPECTOR-HOURS ON SITE IN THE AREAS OF ON SITE FOLLOWUP OF LICENSEE EVENT REPORTS. OF THE AREAS INSPECTED, ONE ITEM OF NONCOMPLIANCE WAS FOUND.

REVIEWED BY OMPA LOEB: R. Bole **DATE:** 12/10/80

REVIEWED BY NRR DL: C R Stahle **DATE:** 12/11/80

SEQUOYAH 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-327
2. Utility Contact: DAVID DUPREE (615) 842-0295
3. Licensed Thermal Power (MWt): 3411
4. Nameplate Rating (Gross MWe): 1220.5
5. Design Electrical Rating (Net MWe): 1140
6. Maximum Dependable Capacity (Gross MWe): 1185
7. Maximum Dependable Capacity (Net MWe): 1140
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	745.0	5,881.0	5,881.0
12. Hours Reactor Was Critical	143.6	685.6	685.4
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	18.0	35.4	36.4
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	29,309	68,668	68,668
17. Gross Electrical Energy (MWH)	4,689	4,719	4,719
18. Net Electrical Energy (MWH)	0	0	0
19. Unit Service Factor			
20. Unit Availability Factor			
21. Unit Capacity Factor (MDC Net)			
22. Unit Capacity Factor (DER Net)			
23. Unit Forced Outage Rate			

NOT IN COMMERCIAL OPERATION

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause and Corrective Action to Prevent Recurrence
	07/23/80	F	727.0	H	4				POWER ASCENSION TESTING - THE UNIT HAD SEVEN REACTOR TRIPS IN THE MONTH OF OCTOBER. THE ELECTRICAL GENERATOR WAS SYNCHRONIZED TO THE GRID FOR THE FIRST TIME THIS MONTH ON OCTOBER 31.
8	10/02/80	F	3.3	G	3				REACTOR TRIPPED ON LOW FEEDWATER LEVEL.
9	10/12/80	F	3.1	A	3				MAIN FEEDWATER PUMP 1A TRIPPED ON LOW VACUUM WHICH RESULTED IN A REACTOR TRIP.
10	10/12/80	F	2.7	A	3				REACTOR TRIPPED ON LOW STEAM GENERATOR LEVEL WHEN THE #3 MAIN FEEDWATER BYPASS VALVE DID NOT OPERATE.
11	10/13/80	F	3.1	A	3				REACTOR TRIPPED ON HIGH STEAM GENERATOR LEVEL AFTER OPERATOR HAD OVERFED STEAM GENERATOR.
12	10/13/80	F	2.5	H	3				MAIN FEEDWATER PUMP 1A TRIPPED ON LOW VACUUM WHICH RESULTED IN A REACTOR TRIP.
13	10/30/80	F	2.8	H	3				REACTOR TRIPPED ON HIGH STEAM GENERATOR LEVEL AFTER OPERATOR HAD OVERFED STEAM GENERATOR.
14	10/30/80	F	1.6	H	3				REACTOR TRIPPED ON LOW STEAM GENERATOR LEVEL WHILE INCREASING POWER.

SUMMARY

SEQUOYAH IS IN POWER ASCENSION AND ANTICIPATING COMMERCIAL OPERATION ON DECEMBER 30, 1980.

LEGEND

[1]Type:
F--Forced
S--Scheduled

[2]Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & Licensee Examination
F--Administrative
G--Operational Error
H--Other

[3]Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4]Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5]Exhibit H:
Same source

ST. LUCIE 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Florida/St. Lucie 12 miles SE of Ft. Pierce
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	PWR
4	Power Level (MWT)	2560
	Design Electrical Rating (MWE Net)	802
6.	Date of Initial Criticality	April 22, 1976
7.	Date of Commercial Operation	December 21, 1976
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Atlantic Ocean

Utility & Contractor Information

10.	Licensee	Florida Power & Light Co.
11.	Corporate Address	9250 W. Flagler Street P. O. Box 529100 Miami, Florida 33152
12.	Corporate Contact	R. E. Uhrig, V.P.
13.	Architect/Engineer	Ebasco
14.	Nuclear Steam Supply System	Combustion Engineering
15.	Constructor	Ebasco

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. LEWIS
18.	IE Section Chief	R. A. Martin
19.	IE Resident Inspector	S. A. ELROD
20.	Licensing Project Manager	C. Nelson
21.	Docket Number	50-335
22.	License Number/Date of Issue	DPR-67, March 1, 1976
23.	Public Document Room	Indian River Junior College Library 3209 Virginia Avenue Ft. Pierce, Florida 33450

INSPECTION STATUS

ENFORCEMENT STATUS

INADEQUATE PROCEDURES FOR SEARCHING PACKAGES.
(8029 INFRACTION)

FAILURE TO POSITIVELY CONTROL ALL POINTS OF PERSONNEL AND VEHICLE ACCESS INTO VITAL AREAS.
(8029 VIOLATION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

NORMAL OPERATION.

+ LAST IE SITE INSPECTION DATE: SEPTEMBER 28 - NOVEMBER 7, 1980

+ INSPECTION REPORT NO: 50-335/80-35

SUMMARY

INSPECTION SUMMARY

+ INSPECTION AUGUST 27-29 (80-28): THIS SPECIAL, UNANNOUNCED MANAGEMENT INSPECTION INVOLVED 5 INSPECTOR-HOURS AT THE GENERAL OFFICES AND ONSITE IN THE AREA OF FOLLOWUP OF NONCOMPLIANCE RELATIVE TO 10 CFR PART 21 PROCEDURES. NONCOMPLIANCE 335/79-33-01, "10 CFR PART 21 PROCEDURES" ARE CLOSED.

INSPECTION SEPTEMBER 17-19 (80-31): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 34 INSPECTOR-HOURS ON SITE IN THE AREAS OF LICENSEE ACTION ON PREVIOUS INSPECTIONS FINDINGS, EQUIPMENT STORAGE, WELDING FILLER MATERIAL CONTROL, CHLORIDE CONTROL, REACTOR COOLANT PRESSURE BOUNDARY PIPING - OBSERVATION OF WORK AND WORK ACTIVITIES, AND REVIEW OF WELDER QUALIFICATION. OF THE SIX AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN FOUR AREAS; ONE ITEM OF NONCOMPLIANCE WAS FOUND IN EACH OF TWO AREAS (INFRACTION - "IMPROPER VALVE STORAGE"; AND INFRACTION - "WELDING FILLER MATERIAL CONTROL").

INSPECTION SEPTEMBER 4-7 (80-35): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 28 INSPECTOR-HOURS ON SITE IN THE AREAS OF BULLETIN, CIRCULAR, AND INFORMATION NOTICE REVIEW, REVIEW OF ITEMS FROM PREVIOUS INSPECTOR REPORTS, AND REVIEW OF PLANT OPERATIONS. OF THE THREE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

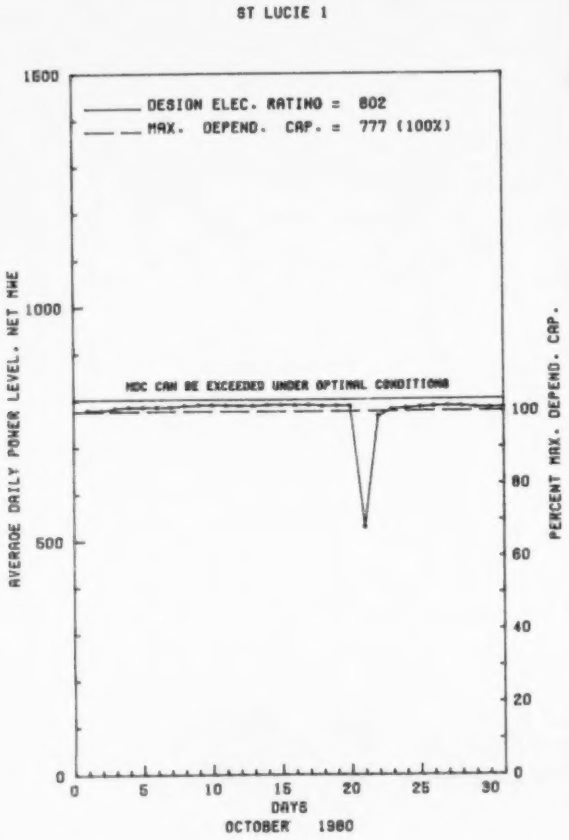
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-033/ 03L-0	07/24/80	08/25/80	DROPPED CONTROL ROD DUE TO MAINTENANCE SHORTING TEST POINT.
80-034/ 03L-0	08/01/80	09/02/80	FAILURE OF POWER SUPPLY CAUSED CEA TO DROP TO FULLY INSERTED POSITION.
80-035/ 03L-0	08/04/80	09/03/80	FAILURE OF POWER SUPPLY CAUSED CEA TO DROP TO FULLY INSERTED POSITION.
80-036/ 03L-0	08/05/80	09/03/80	FAILURE OF POWER SUPPLY CAUSED CEA TO DROP TO FULLY INSERTED POSITION.
80-039/ 03L-0	08/14/80	09/15/80	DOSE EQUIVALENT IODINE I-131 EXCEEDED TECHNICAL LIMIT 3.4.8.A A LIMIT OF 1.0 UCI/GRAM.
80-040/ 03L-0	08/15/80	09/15/80	DURING STARTUP MODE A CEA WAS DROPPED TO THE FULLY INSERTED POSITION.
80-041/ 03L-0	08/14/80	09/15/80	PACKING FAILED ON V1236 PRESSURIZER MINI-SPRAY VALVE CAUSING COOLANT LEAKAGE TO EXCEED TS 3.4.6.2.D.
80-042/ 03L-0	08/21/80	09/19/80	DROPPED CEA.
80-043/ 03L-0	08/21/80	09/18/80	DROPPED CEA.
80-044/ 03L-0	08/24/80	09/18/80	DIGITAL DATA PROCESSING SYSTEM WHICH MONITORS INCORE NEUTRON DETECTORS FAILED.

REVIEWED BY OMPA LOEB: MR Beebe **DATE:** 12/10/80

REVIEWED BY NRR DL: C Nelson **DATE:** 12/11/80

ST. LUCIE 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-335
2. Utility Contact: V. T. Chilson (305) 552-3824
3. Licensed Thermal Power (MWt): 2560
4. Nameplate Rating (Gross MWe): 1000 x 0.85 = 850
5. Design Electrical Rating (Net MWe): 802
6. Maximum Dependable Capacity (Gross MWe): 822
7. Maximum Dependable Capacity (Net MWe): 777
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	745.0	7,320.0	33,864.0
12. Hours Reactor Was Critical	740.3	5,457.8	26,967.7
13. Reactor Reserve Shutdown Hours	.0	.0	129.4
14. Hours Generator On-Line	739.2	5,345.4	26,175.5
15. Unit Reserve Shutdown Hours	.0	7.3	39.3
16. Gross Thermal Energy (MWH)	1,885,676	13,271,993	63,521,559
17. Gross Electrical Energy (MWH)	612,420	4,299,700	20,594,200
18. Net Electrical Energy (MWH)	579,456	4,048,350	19,376,246
19. Unit Service Factor	99.2	73.0	77.3
20. Unit Availability Factor	99.2	73.1	77.4
21. Unit Capacity Factor (MDC Net)	100.1	71.2	73.6
22. Unit Capacity Factor (DER Net)	97.0	69.0	71.3
23. Unit Forced Outage Rate	.8	8.4	6.0

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
17	10/21/80	F	5.8	A	3		HH	INSTRU	UNIT WAS TRIPPED BY THE STEAM GENERATOR LEVEL PROTECTION SYSTEM CAUSED BY A SPURIOUS CONTROL SIGNAL TO FEEDWATER CONTROL VALVE NO. 1A.

SUMMARY

ST. LUCIE 1 OPERATED AT OR ABOVE FULL POWER DURING OCTOBER, WITH ONE OUTAGE DUE TO EQUIPMENT FAILURE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuation
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source

SURRY 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1. Location (state/county) (nearest population center)	Virginia/Surry 17 miles NW of Newport News
2. Electric Reliability Council	Southeastern Electric Reliability Council
3. Type of Reactor	PWR
4. Power Level (MWT)	2441
5. Design Electrical Rating (MWE Net)	822
6. Date of Initial Criticality	July 1, 1972
7. Date of Commercial Operation	December 22, 1972
8. Condenser Cooling Method	Once Through
9. Condenser Cooling Water Source	James River

Utility & Contractor Information

10 Licensee	Virginia Electric and Power Company
11. Corporate Address	P. O. Box 26666 Richmond, Virginia 23261
12. Corporate Contact	J. H. Ferguson Exec. V.P. - Power
13. Architect/Engineer	Stone & Webster
14. Nuclear Steam Supply System	Westinghouse
15. Constructor	Stone & Webster

Regulatory Information

16. IE Region responsible	II
17. IE Branch Chief	R.C. Lewis
18. IE Section Chief	P. J. Kellogg
19. IE Resident Inspector	D. Burke (804) 357-2101
20. Licensing Project Manager	D. Neighbors
21. Docket Number	50-280
22. License Number/Date of Issue	DPR-32, 5/25/72
23. Public Document Room	Swem Library College of William and Mary Williamsburg, Virginia 23185

INSPECTION STATUS

ENFORCEMENT STATUS

AS REQUIRED BY CRITERION V OF APPENDIX B TO 10 CFR 50 AND IMPLEMENTED BY VEPCO TOPICAL REPORT VEP-1-A SECTION 17 PARAGRAPH 17.2.5, STATES IN PART THAT...PROCEDURES SHALL INCLUDE APPROPRIATE QUANTITATIVE ACCEPTANCE CRITERIA FOR DETERMINING THAT IMPORTANT ACTIVITIES HAVE BEEN SATISFACTORILY ACCOMPLISHED. CONTRARY TO THE ABOVE, ON SEPTEMBER 10, 1980,NDT-15.1 "VISUAL EXAMINATION OF NUCLEAR COMPONENTS) DID NOT CONTAIN SPECIFIC ACCEPTANCE CRITERIA FOR INITIAL INSPECTION OF WELDS FABRICATED TO APPLICABLE CONSTRUCTION CODES.
(8035 INFRACTION)

AS REQUIRED BY CRITERION XVII OF APPENDIX B TO 10 CFR 50 AND IMPLEMENTED BY VEPCO TOPICAL REPORT VEP-1-A SECTION 17 PARAGRAPH 17.2.17, STATES IN PART THAT QUALITY ASSURANCE RECORDS RELATING TO...DOCUMENTARY EVIDENCE OF...ACTIVITIES AFFECTING QUALITY...INCLUDE MONITORING OF WORK PERFORMANCE, ARE MAINTAINED AND ARE RETRIEVABLE. CONTRARY TO THE ABOVE, ON SEPTEMBER 10, 1980, THERE WAS NO DOCUMENTARY EVIDENCE TO VERIFY THAT VEPCO'S NDT GROUP HAD MONITORED VENDOR WORK PERFORMANCE RELATIVE TO IEB 79-17 NDE INSPECTIONS.
(8035 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

STEAM GENERATOR TUBE LEAKS.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

- UNIT IS CURRENTLY SHUTDOWN FOR STEAM GENERATOR REPLACEMENT.
- LAST IE SITE INSPECTION DATE: SEPTEMBER 10, 1980

INSPECTION REPORT NO: 50-280/80-35

SUMMARY

INSPECTION SUMMARY

- INSPECTION JULY 7 - AUGUST 29 (80-34): THIS ROUTINE INSPECTION BY THE RESIDENT INSPECTOR INVOLVED 95 INSPECTOR-HOURS ON SITE IN THE AREAS OF PLANT OPERATIONS AND OPERATING RECORDS, PLANT MODIFICATIONS, MAINTENANCE AND TESTING, LICENSEE EVENT REPORTS, AND PLANT SECURITY. OF THE FOUR AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE FOUND IN THE THREE AREAS; FOUR ITEMS OF NONCOMPLIANCE WERE FOUND IN ONE AREA (INFRACTION - FAILURE TO MAINTAIN CERTAIN SI INSTRUMENTATION OPERABLE AS REQUIRED BY TS 3.7; INFRACTION - FAILURE TO FOLLOW PROCEDURES FOR REMOVING SAFETY RELATED EQUIPMENT FROM SERVICE; DEFICIENCY - FAILURE TO REPORT THE INOPERABLE SI INSTRUMENTATION TO THE NRC WITHIN ONE HOUR IN ACCORDANCE WITH 10 CFR 50.72 (A)(6); AND INFRACTION - FAILURE TO REVISE INSTRUMENT SURVEILLANCE PROCEDURES FOLLOWING INSTRUMENT SETPOINT CHANGES). OF THE FOUR NONCOMPLIANCES THE LICENSEE DID NOT AGREE WITH THE DEFICIENCY - FAILURE TO REPORT THE INOPERABLE SI INSTRUMENTATION TO THE NRC WITHIN ONE HOUR IN ACCORDANCE WITH 10 CFR 50.72(A)(6).

INSPECTION SEPTEMBER 9-10 (80-35): THIS ROUTINE UNANNOUNCED INSPECTION INVOLVED 5 INSPECTOR-HOURS ON SITE IN AREAS OF PREVIOUS INSPECTION FINDINGS (UNITS 1 AND 2), IE BULLETIN 79-17 REV. 1, RECORD REVIEW, (UNITS 1 AND 2). OF THE TWO AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN ONE AREA. TWO ITEMS OF NONCOMPLIANCE WERE FOUND IN ONE AREA (DEFICIENCY - FAILURE TO DOCUMENT QA SURVEILLANCE ACTIVITIES; INFRACTION - VISUAL INSPECTION OF WELDS WITHOUT DOCUMENTED PROCEDURE).

REPORTS RECEIVED FROM LICENSEE

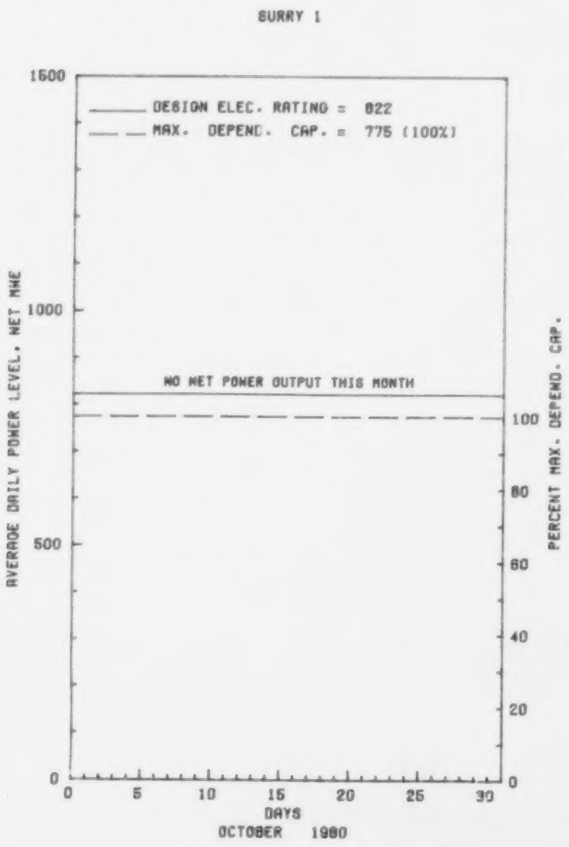
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-060/ 03L-0	08/31/80	09/26/80	LOW FLOW ALARM ON RAD MONITOR FOR PROCESS VENT SYSTEM.
80-066/ 03L-0	09/02/80	10/02/80	EMERGENCY DIESEL FUEL TANK NOT FILLING AUTOMATICALLY.

REVIEWED BY OMPA LOEB: ________ **DATE:** 12/10/80

REVIEWED BY NRR DL: Don Neighbors **DATE:** 12/11/80

SURRY 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-280
2. Utility Contact: SUE DUNN (804) 357-3184
3. Licensed Thermal Power (MWt): 2441
4. Nameplate Rating (Gross MWe): 942 x 0.9 = 848
5. Design Electrical Rating (Net MWe): 822
6. Maximum Dependable Capacity (Gross MWe): 811
7. Maximum Dependable Capacity (Net MWe): 775
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	745.0	7,320.0	68,904.0
12. Hours Reactor Was Critical	0	3,786.9	42,538.5
13. Reactor Reserve Shutdown Hours	0	182.6	3,721.5
14. Hours Generator On-Line	0	3,763.8	41,668.8
15. Unit Reserve Shutdown Hours	0	63.6	3,736.2
16. Gross Thermal Energy (MWH)	0	8,674,529	96,389,401
17. Gross Electrical Energy (MWH)	0	2,621,245	31,301,743
18. Net Electrical Energy (MWH)	0	2,473,025	29,659,924
19. Unit Service Factor	0	51.4	60.5
20. Unit Availability Factor	0	52.3	65.9
21. Unit Capacity Factor (MDC Net)	0	43.6	55.6
22. Unit Capacity Factor (DER Net)	0	41.1	52.4
23. Unit Forced Outage Rate	0	39.1	26.4

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 09/01/81

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-14	09/14/80	S	745.0	A	4				CONTINUATION OF SHUTDOWN FOR STEAM GNERATOR REPLACEMENT OUTAGE WHICH COMMENCED 09/14/80.

SUMMARY

SURRY 1 REMAINED SHUTDOWN DURING OCTOBER FOR STEAM GENERATOR REPLACEMENT.

LEGENC

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maint.nance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

SURRY 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Virginia/Surry 17 miles NW of Newport News
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2441
5.	Design Electrical Rating (MWE Net)	822
6.	Date of Initial Criticality	March 7, 1973
7.	Date of Commercial Operation	May 1, 1973
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	James River

Utility & Contractor Information

10.	Licensee	Virginia Electric and Power Company
11.	Corporate Address	P. O. Box 26666 Richmond, Virginia 23261
12.	Corporate Contact	J. H. Ferguson Exec. V.P. - Power
13.	Architect/Engineer	Stone & Webster
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R.C. Lewis
18.	IE Section Chief	P. J. Kellog
19.	IE Resident Inspector	D. Burke (804) 357-2101
20.	Licensing Project Manager	D. Neighbors
21.	Docket Number	50-281
22.	License Number/Date of Issue	DPR-37, 1/29/73
23.	Public Document Room	Swem Library College of William and Mary Williamsburg, Virginia 23185

INSPECTION STATUS

ENFORCEMENT STATUS

AS REQUIRED BY 10CFR50.72(A)(B), PERSONNEL ERRORS OR PROCEDURAL INADEQUACIES WHICH, DURING NORMAL OPERATIONS, ANTICIPATED OPERATIONAL OCCURRENCES, OR ACCIDENT CONDITIONS, PREVENTS OR COULD PREVENT, BY ITSELF, THE FULFILLMENT OF THE SAFETY FUNCTION OF THOSE STRUCTURES, SYSTEMS, AND COMPONENTS IMPORTANT TO SAFETY, MUST BE REPORTED TO THE NRC OPERATIONS CENTER BY TELEPHONE AS SOON AS POSSIBLE AND IN ALL CASES WITHIN ONE HOUR OF THE OCCURRENCE. CONTRARY TO THE ABOVE, WHEN THE MAIN FLOW INSTRUMENTATION WAS FOUND TO BE INOPERABLE AT 12:30 A.M. ON AUGUST 19, 1980, THE NRC OPERATIONS CENTER WAS NOT NOTIFIED WITHIN ONE HOUR.
(8037 DEFICIENCY)

AS REQUIRED BY TECHNICAL SPECIFICATION TABLE 3.7-2, ITEM I.E., "ENGINEERING SAFEGUARDS ACTION", THE HIGH STEAM LINE FLOW INSTRUMENTATION FOR ACTUATION OF SAFETY INJECTION SHALL BE OPERABLE WHEN THE REACTOR IS ABOVE THE COLD SHUTDOWN CONDITION. CONTRARY TO THE ABOVE, ON AUGUST 19, 1980, DURING OPERATION AT 12% POWER, THE SIX MAIN STEAMLINE FLOW INSTRUMENTS WERE NOT OPERABLE, IN THAT THE FLOW TRANSMITTERS WERE ISOLATED AND THE ELECTRICAL FUSES WERE REMOVED.
(8037 INFRACTION)

AS REQUIRED BY TECHNICAL SPECIFICATION 6.4.A, DETAILED WRITTEN PROCEDURES WITH APPROPRIATE CHECK-OFF LISTS AND INSTRUCTIONS SHALL BE PROVIDED FOR PLANT OPERATION AND ALL SYSTEMS AND COMPONENTS INVOLVING NUCLEAR SAFETY OF THE STATION, AND FOR CALIBRATION AND TESTING OF INSTRUMENTS, COMPONENTS, AND SYSTEMS INVOLVING NUCLEAR SAFETY OF THE STATION. CONTRARY TO THE ABOVE, PERIODIC TEST PROCEDURE NO. 36, "INSTRUMENT SURVEILLANCE" WAS NOT REVISED PRIOR TO UNIT 2 STARTUP ON AUGUST 14, 1980, REFLECT TECHINCAL SPECIFICATIONS 3.4.2.A.3 AND 3.4.2.A TO THE REQUIRED LEVELS IN THE REFUELING WATER STORAGE TANK (RWST) AND THE CHEMICAL ADDITION TANK (CAT). THE REVISED TS MINIMUM LEVELS OF 96% AND 97% FOR THE RWST AND CAT WERE DENOTED IN THE SHIFT ORDER BOOK, BUT THE MINIMUM RWST AND CAT LEVELS LISTED IN PT 36 WERE 94% AND 77%, RESPECTIVELY, AND LED TO OPERATION OF THE RWST AT 95% AND THE CAT AT 96% ON AUGUST 14 AND 15, 1980.
(8037 INFRACTION)

AS REQUIRED BY TECHNICAL SPECIFICATION 6.4.D, DETAILED WRITTEN PROCEDURES WITH APPROPRIATE CHECK-OFF LISTS AND INSTRUCTIONS PROVIDED FOR OPERATION, MAINTENANCE, CALIBRATION OR TESTING OF INSTRUMENTS, COMPONENTS, AND SYSTEMS INVOLVING NUCLEAR SAFETY SHALL BE FOLLOWED. VEPCO NPS QA MANUAL, SECTION 14, ASSIGNS RESPONSIBILITY FOR THE TAGGING OF ALL EQUIPMENT TO ASSURE THAT IT IS PERFORMED IN ACCORDANCE WITH PROCEDURES AND IN A SAFE MANNER. CONTRARYTO THE ABOVE, APPROPRIATE TAGGING WAS NOT PERFORMED AND PROCEDURE PT 2.9 AND 2.9A WERE NOT FOLLOWED AS REQUIRED WHEN ELECTRICAL FUSES WERE REMOVED AND THE TRANSMITTERS ISOLATED ON THE SIX UNIT 2 MAIN STEAMLINE FLOW INSTRUMENTS FI 2472, 2475, 2484, 2485, 2494 PRIOR TO UNIT 2 STARTUP. THE STEAM FLOW INSTRUMENTS WERE FOUND ISOLATED AND INOPERABLE AT 12:30 A.M. ON AUGUST 19, 1980, WHILE OPERATING WAS AT 12% POWER.
(8037 INFRACTION)

AS REQUIRED BY CRITERION V OF APPENDIX B TO 10 CFR 50 AND IMPLEMENTED BY VEPCO TOPICAL REPORT VEP-1-A SECTION 17 PARAGRAPH 17.2.5, STATES IN PART THAT...PROCEDURES SHALL INCLUDE APPROPRIATE QUANTITATIVE ACCEPTANCE CRITERIA FOR DETERMINING THAT IMPORTANT ACTIVITIES HAVE BEEN SATISFACTORILY ACCOMPLISHED. CONTRARY TO THE ABOVE, ON SEPTEMBER 10, 1980,NDT-15.1 "VISUAL EXAMINATION OF NUCLEAR COMPONENTS) DID NOT CONTAIN SPECIFIC ACCEPTANCE CRITERIA FOR INITIAL INSPECTION OF WELDS FABRICATED TO APPLICABLE CONSTRUCTION CODES.
(8038 INFRACTION)

AS REQUIRED BY CRITERION XVII OF APPENDIX B TO 10 CFR 50 AND IMPLEMENTED BY VEPCO TOPICAL REPORT VEP-1-A SECTION 17 PARAGRAPH 17.2.17, STATES IN PART THAT QUALITY ASSURANCE RECORDS RELATING TO...DOCUMENTARY EVIDENCE OF...ACTIVITIES AFFECTING QUALITY...INCLUDE MONITORING OF WORK PERFORMANCE, ARE MAINTAINED AND ARE RETRIEVABLE. CONTRARY TO THE ABOVE, ON SEPTEMBER 10, 1980, THERE WAS NO DOCUMENTARY EVIDENCE TO VERIFY THAT VEPCO'S NDT GROUP HAD MONITORED VENDOR WORK PERFORMANCE RELATIVE TO IEB 79-17 NDE INSPECTIONS.
(8038 DEFICIENCY)

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-024/ 03L-0	08/28/80	09/26/80	CONTAINMENT ISOLATION VALVE TV-SS-200B FAILED.

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

SYSTEM MODIFICATIONS AND REANALYSIS FOR SEISMIC SUPPORTS ARE CONTINUING.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

- UNIT IS CURRENTLY AT 100% POWER.
- LAST IE SITE INSPECTION DATE: SEPTEMBER 10, 1980
- INSPECTION REPORT NO: 50-281/80-38

SUMMARY

INSPECTION SUMMARY

- INSPECTION JULY 7 - AUGUST 29 (80-37): THIS ROUTINE INSPECTION BY THE RESIDENT INSPECTOR INVOLVED 95 INSPECTOR-HOURS ON SITE IN THE AREAS OF PLANT OPERATIONS AND OPERATING RECORDS, PLANT MODIFICATIONS, MAINTENANCE AND TESTING, LICENSEE EVENT REPORTS, AND PLANT SECURITY. OF THE FOUR AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE FOUND IN THE THREE AREAS; FOUR ITEMS OF NONCOMPLIANCE WERE FOUND IN ONE AREA (INFRACTION - FAILURE TO MAINTAIN CERTAIN SI INSTRUMENTATION OPERABLE AS REQUIRED BY TS 3.7; INFRACTION - FAILURE TO FOLLOW PROCEDURES FOR REMOVING SAFETY RELATED EQUIPMENT FROM SERVICE; DEFICIENCY - FAILURE TO REPORT THE INOPERABLE SI INSTRUMENTATION TO THE NRC WITHIN ONE HOUR IN ACCORDANCE WITH 10 CFR 50.72 (A)(6); AND INFRACTION - FAILURE TO REVISE INSTRUMENT SURVEILLANCE PROCEDURES FOLLOWING INSTRUMENT SETPOINT CHANGES). OF THE FOUR NONCOMPLIANCES THE LICENSEE DID NOT AGREE WITH THE DEFICIENCY - FAILURE TO REPORT THE INOPERABLE SI INSTRUMENTATION TO THE NRC WITHIN ONE HOUR IN ACCORDANCE WITH 10 CFR 50.72(A)(6).

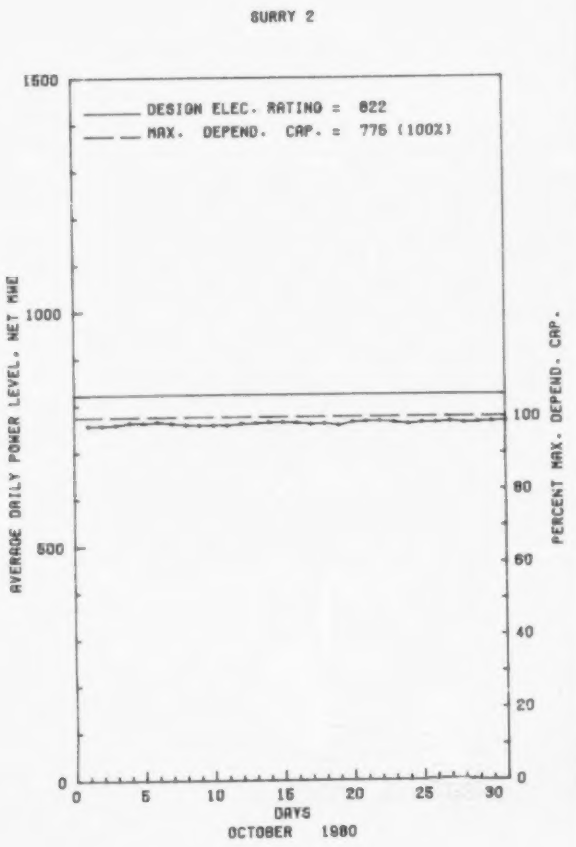
INSPECTION SEPTEMBER 9-10 (80-38): THIS ROUTINE UNANNOUNCED INSPECTION INVOLVED 4 INSPECTOR-HOURS ON SITE IN AREAS OF PREVIOUS INSPECTION FINDINGS (UNITS 1 AND 2), IE BULLETIN 79-17 REV. 1, RECORD REVIEW, (UNITS 1 AND 2). OF THE TWO AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN ONE AREA. TWO ITEMS OF NONCOMPLIANCE WERE FOUND IN ONE AREA (DEFICIENCY - FAILURE TO DOCUMENT QA SURVEILLANCE ACTIVITIES; INFRACTION - VISUAL INSPECTION OF WELDS WITHOUT DOCUMENTED PROCEDURE).

REVIEWED BY OMPA LOEB: ______ **DATE:** 12/10/80

REVIEWED BY NRR DL: Don Neighbors **DATE:** 12/11/80

SURRY 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-281
2. Utility Contact: SUE DUNN (804) 357-3184
3. Licensed Thermal Power (MWt): 2441
4. Nameplate Rating (Gross MWe): 942 x 0.9 = 848
5. Design Electrical Rating (Net MWe): 822
6. Maximum Dependable Capacity (Gross MWe): 811
7. Maximum Dependable Capacity (Net MWe): 775
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	745.0	7,320.0	65,784.0
12. Hours Reactor Was Critical	745.0	1,828.7	36,328.6
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	745.0	1,690.7	35,686.9
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,814,032	3,733,719	82,927,802
17. Gross Electrical Energy (MWH)	598,060	1,209,120	27,077,965
18. Net Electrical Energy (MWH)	567,250	1,139,888	25,676,493
19. Unit Service Factor	100.0	23.1	54.2
20. Unit Availability Factor	100.0	23.1	54.2
21. Unit Capacity Factor (MDC Net)	98.2	20.1	50.4
22. Unit Capacity Factor (DER Net)	92.6	18.9	47.5
23. Unit Forced Outage Rate	.0	4.9	20.4

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
APRIL 1, 1981 - 3 DAYS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
NONE									

SUMMARY

SURRY 2 OPERATED AT NEAR FULL POWER DURING OCTOBER, WITH NO OUTAGES OR REDUCTIONS REPORTED.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source

THREE MILE ISLAND 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Pennsylvania/Dauphin 10 miles SE of Harrisburg
2.	Electric Reliability Council	Mid-Atlantic Area Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2535
5.	Design Electrical Rating (MWE Net)	819
6.	Date of Initial Criticality	June 5, 1974
7.	Date of Commercial Operation	September 2, 1974
8.	Condenser Cooling Method	Closed
9.	Condenser Cooling Water Source	Susquehanna River

Utility & Contractor Information

10.	Licensee	Metropolitan Edison Company
11.	Corporate Address	P. O. Box 542 Reading, Pennsylvania
12.	Corporate Contact	R. C. Arnold, Sr. Vice-President 260 Cherry Hill Rd. Parsippany, NJ 07054
13.	Architect/Engineer	Gilbert Associates, Inc.
14.	Nuclear Steam Supply System	B&W
15.	Constructor	UE&C

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Section Chief	A. N. Fasano (717) 948-1123 (FTS) 590-1123
19.	NRC-TMI Support Staff	D. Haverkamp (717) 948-1127 (FTS) 590-1127
20.	Licensing Project Manager	D. C. Dilanni / H. Silver (X27435)
21.	Docket Number	50-289
22.	License Number/Date of Issue	DPR-50, 4/19/74
23.	Public Document Room	Government Publications Section State Library of Pennsylvania Education Building Commonwealth and Walnut Street Harrisburg, Pennsylvania 17126

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO 10 CFR 50.59(B), CHANGES MADE PURSUANT TO 10 CFR 50.59(A) DURING CALENDAR YEARS 1978 AND 1979 WERE NOT REPORTED TO THE NRC.
(8021 DEFICIENCY)

CONTRARY TO T.S. 3.18.4.1 AND T.S. 3.18.4.2, WHEN THE CO2 SYSTEM FOR THE CABLE SPREADING ROOM BECAME INOPERABLE WHILE IN COLD SHUTDOWN, A CONTINUOUS FIRE WATCH WAS ESTABLISHED WITHIN ONE HOUR, BUT NOT MAINTAINED PRIOR TO RESTORING THE SYSTEM TO AN OPERABLE CONDITION.
(8021 INFRACTION)

CONTRARY TO T.S. 6.5.1.6, DURING THE PAST TWELVE MONTHS, PORC DID NOT REVIEW AND REPORT THEIR EVALUATION AND RECOMMENDATIONS FOR VIOLATIONS OF TECHNICAL SPECIFICATIONS OR REVIEW VIOLATIONS OF INTERNAL STATION PROCEDURES HAVING NUCLEAR SAFETY SIGNIFICANCE AS REPORTED IN QUALITY ASSURANCE AUDIT REPORTS. CONTRARY TO T.S. 6.5.1.7, DURING THE PAST TWELVE MONTHS, PORC DID RENDER DETERMINATIONS WITH REGARD TO WHETHER OR NOT ITEMS CONSIDERED UNDER TECHNICAL SPECIFICATION 6.5.1.6.(E), INCLUDING REPORTABLE OCCURRENCES, VIOLATIONS OF TECHNICAL SPECIFICATIONS, AND VIOLATIONS OF INTERNAL STATION PROCEDURES, CONSTITUTED AN UNREVIEWED SAFETY QUESTION.
(8021 INFRACTION)

CONTRARY TO T.S. 6.5.1, DURING PORC MEETINGS 80-06 AND 80-11, THE QUORUM TOOK CREDIT FOR TWO ALTERNATE MEMBERS.
(8021 DEFICIENCY)

CONTRARY TO T.S. 6.5.2.B.1 AND 6.5.2.B.7, THE GENERAL OFFICE REVIEW BOARD DID NOT REVIEW VIOLATIONS OF TECHNICAL SPECIFICATIONS OR OPERATING LICENSE DPR-50 AS REPORTED IN QUALITY ASSURANCE AUDIT REPORTS, DURING THE PERIOD APRIL 1, 1979 TO JUNE 30, 1980.
(8021 INFRACTION)

CONTRARY TO T.S. 6.5.2.A.2, FOLLOWING THE TMI-2 ACCIDENT OF MARCH 28, 1979, THE MET-ED CORPORATE TECHNICAL SUPPORT STAFF REVIEW RESPONSIBILITIES HAD NOT BEEN ASSUMED IN WRITING BY ANY OTHER ORGANIZATIONAL UNIT FOR: (1) THE REVIEW OF PROPOSED CHANGES TO THE TECHNICAL SPECIFICATIONS AND OPERATING LICENSE DPR-50, AND (2) THE REVIEW OF REPORTABLE OCCURRENCES, OTHER THAN THOSE REQUIRING TWENTY-FOUR HOUR WRITTEN NOTIFICATION TO THE NRC.
(8021 DEFICIENCY)

CONTRARY TO THE PHYSICAL SECURITY PLAN, PARAGRAPHS I(C)(3) AND 3.1.1 AN ADEQUATE ISOLATION ZONE WAS NOT MAINTAINED ON BOTH SIDES OF THE PROTECTED AREA PHYSICAL BARRIER.
(8021 INFRACTION)

CONTRARY TO THE PHYSICAL SECURITY PLAN, PARAGRAPH (I)(D)(1), THE SPO RESPONSIBLE FOR THE LAST ACCESS CONTROL FUNCTION WAS NOT ADEQUATELY ENCLOSED WITHIN A BULLET-RESISTING STRUCTURE.
(8021 INFRACTION)

CONTRARY TO THE PHYSICAL SECURITY PLAN, PARAGRAPHS I(D)(7) AND I(E)(2), THE VITAL AREA WAS NOT PROTECTED BY AN ACTIVE INTRUSION ALARM SYSTEM IN TWO LOCATIONS.
(8021 INFRACTION)

FAILURE TO MAINTAIN ISOLATION ZONE AS REQUIRED.
(8024 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

COMPLETE REQUIRED MODIFICATIONS PRIOR TO RESTART.

PLANT STATUS:

THE PLANT REMAINS SHUTDOWN, BY NRC ORDER, PENDING COMPLETION OF MODIFICATIONS AND OTHER ACTIONS RELATED TO THE TMI-2 ACCIDENT.

♦ LAST IE SITE INSPECTION DATE: 9/2 - 10/31/80

♦ INSPECTION REPORT NO: 50-289/80-29

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-003/ DER	06/13/80	06/19/80	IWTS PH WAS 5.4 WHICH WAS OUTSIDE THE 6.0-9.0 RANGE ALLOWED BY THE NPDES DISCHARGE PERMIT NO. 9920
80-013/ 03L	07/15/80	08/18/80	DURING SETPOINT VERIFICATION SURVEILLANCE TESTING, PRESSURIZER CODE SAFETY VALVE RC-RV1B LIFTED AT 2330 PSIG, WHICH WAS NOT WITHIN LIMITS OF 2500 PSIG PLUS OR MINUS ONE PERCENT
80-014/ 03L	07/15/80	08/14/80	DURING SURVEILLANCE TESTING, SNUBBER NSE-151 WAS FOUND TO HAVE A LOW FLUID LEVEL AND FAILED THE FUNCTIONAL TEST, PROBABLY DUE TO LEAKAGE PAST A DAMAGED SEAT

SUMMARY

INSPECTION SUMMARY

♦ 50-289/80-18 - JUL 1: SPECIAL CORPORATE MANAGEMENT MEETING TO DISCUSS THE OFFICE OF INSPECTION AND ENFORCEMENT LICENSEE MANAGEMENT REVIEWS/APPRAISALS WHICH WILL AUGMENT THE ROUTINE NRC INSPECTION PROGRAM.

♦ 50-289-80-21 - JUL 7-11, 14-18, 27-31 & AUG 1: A SPECIAL ANNOUNCED PERFORMANCE APPRAISAL BRANCH INSPECTION WAS MADE OF THE LICENSEE'S MANAGEMENT CONTROLS OVER SELECTED LICENSED ACTIVITIES. THE INSPECTION (BY FIVE NRC INSPECTORS) INVOLVED 516 INSPECTOR HOURS AT THE SITE AND IN THE CORPORATE OFFICE. THE LICENSEE'S MANAGEMENT CONTROLS FOR 11 AREAS WERE REVIEWED, AND CONCLUSIONS WERE DRAWN IN EACH AREA BASED ON OBSERVATIONS. THE CONCLUSIONS ARE PRESENTED AS GOOD, AVERAGE, OR POOR AS FOLLOWS: COMMITTEE ACTIVITIES - AVERAGE; QUALITY ASSURANCE AUDITS - GOOD; DESIGN CHANGES AND MODS. - AVERAGE; MAINTENANCE - AVERAGE; REVIEW OF LICENSED ACTIVITIES (OPERATIONS) - AVERAGE; CORRECTIVE ACTION SYSTEMS - AVERAGE; TRAINING (NON-LICENSED) - POOR; INSERVICE INSPECTION AND TESTING (PUMPS AND VALVES) - AVERAGE; FIRE PREVENTION AND PROTECTION - AVERAGE; PROCUREMENT - AVERAGE; AND PHYSICAL PROTECTION - AVERAGE. NINE NONCOMPLIANCES WERE IDENTIFIED: FAILURE OF PORC TO REVIEW CERTAIN VIOLATIONS OF INTERNAL PROCEDURES AND THE TS AND TO RENDER CERTAIN DETERMINATIONS; FAILURE OF PORC TO MEET QUORUM REQUIREMENTS; FAILURE TO PERFORM REVIEW FUNCTIONS OF THE MET-ED CORPORATE TECHNICAL SUPPORT STAFF; FAILURE OF GORB TO REVIEW CERTAIN VIOLATIONS OF INTERNAL PROCEDURES AND THE TECHNICAL SPECIFICATIONS; FAILURE TO REPORT CHANGES MADE PURSUANT TO 10 CFR 50.59(B); FAILURE TO MAINTAIN CONTINUOUS FIRE WATCH WHEN THE CABLE SPREADING ROOM CO2 SYSTEM WAS INOPERABLE; FAILURE TO PROTECT THE VITAL AREA BY AN ACTIVE INTRUSION ALARM SYSTEM IN TWO LOCATIONS; FAILURE TO ADEQUATELY ENCLOSE THE SITE PROTECTION OFFICER RESPONSIBLE FOR THE LAST ACCESS CONTROL FUNCTION; AND, FAILURE TO MAINTAIN AN ADEQUATE ISOLATION ZONE ON BOTH SIDES OF THE PROTECTED AREA PHYSICAL BARRIER.

♦ 50-289/80-23 - JUL 1 - AUG 18: INVESTIGATION OF CONCERNS IDENTIFIED IN SWORN DEPOSITIONS OBTAINED BY THE THREE MILE ISLAND ALERT (TMIA) INTERVENOR GROUP IN PREPARATION FOR THE RESTART HEARINGS. NO INFORMATION TO SUBSTANTIATE THE CONCERNS WAS FOUND DURING THE INVESTIGATION AND NO NONCOMPLIANCES WERE IDENTIFIED.

♦ 50-289/80-24 - AUG 25-29: UNANNOUNCED, PHYSICAL SECURITY INSPECTION BY ONE REGION-BASED INSPECTOR (37 HRS) INCLUDED REVIEW OF SECURITY PLAN; SECURITY PROGRAM AUDIT; SECURITY ORGANIZATION; PHYSICAL BARRIERS (PROTECTED AREA AND VITAL AREAS); LIGHTING; ACCESS CONTROLS (IDENTIFICATION, AUTHORIZATION AND BADGING; SEARCH; ESCORTING; VITAL AREAS); ALARM STATIONS; DETECTION AIDS; ASSESSMENT AIDS; COMMUNICATIONS; TESTING AND MAINTENANCE; COMPENSATORY MEASURES; POWER SUPPLY; RESPONSE; LOCKS, KEYS AND COMBINATIONS; RECORDS AND REPORTS. LICENSEE ACTION ON PREVIOUSLY IDENTIFIED NONCOMPLIANCE ITEMS RELATED TO PHYSICAL SECURITY. FOLLOWUP ON IE CIRCULAR 80-09 REGARDING PLANT COMMUNICATIONS SYSTEMS AND FOLLOWUP ON NRC PERFORMANCE APPRAISAL BRANCH INSPECTION. ONE NONCOMPLIANCE WAS IDENTIFIED: FAILURE TO MAINTAIN PHYSICAL BARRIERS (PROTECTED AREA) AS REQUIRED.

♦ 50-289/80-26 - SEP 10-12: ROUTINE, ONSITE, UNANNOUNCED INSPECTION BY ONE REGION-BASED INSPECTOR (24 HRS) OF WORK ACTIVITIES AND QUALITY RECORDS ASSOCIATED WITH THE RESTART PROGRAM. SPECIFIC AREA SELECTED: TASK LM-9, RELOCATION OF PRESSURIZER TRANSMITTERS ABOVE FLOOD LEVEL; TASK RM-13A, EMERGENCY FEEDWATER CONTROLS. NO NONCOMPLIANCES WERE IDENTIFIED.

♦ 50-289/80-27 - SEP 10-30: UNANNOUNCED, ONSITE, INSPECTION BY ONE REGION-BASED INSPECTOR (50 HRS) OF MAINTENANCE ACTIVITIES RELATED TO CONCERNS EXPRESSED IN PRE-HEARING PROCEEDINGS OF ATOMIC SAFETY AND LICENSING BOARD. NO NONCOMPLIANCES WERE IDENTIFIED.

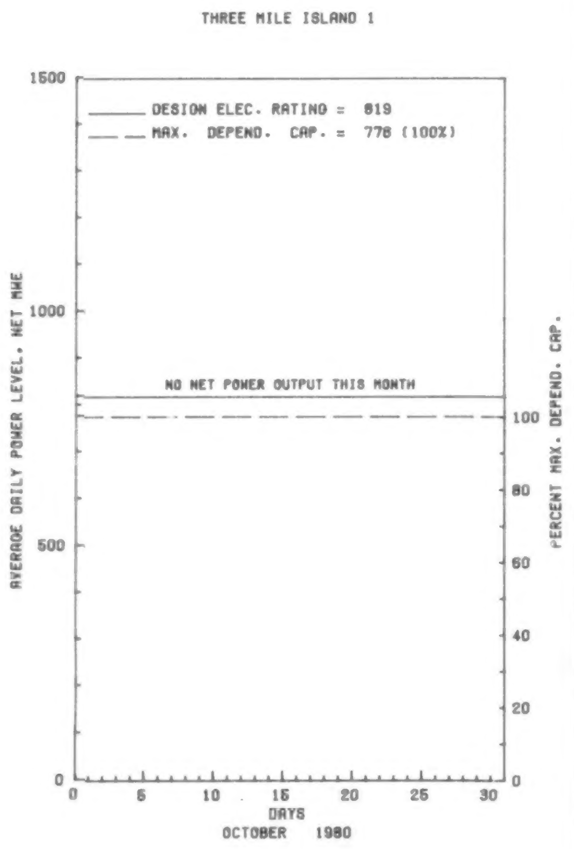
♦ 50-289/80-28 - SEP 23-25: ROUTINE, ONSITE, UNANNOUNCED INSPECTION BY ONE REGION-BASED INSPECTOR (23 HRS) OF INSTALLATIONS ASSOCIATED WITH MODIFICATIONS TO THE PLANT FIRE PROTECTION PROGRAM UNDER TASK NM-40 INCLUDING FIRE DETECTORS, HALON SYSTEM, FIRE DOORS, HOSE STATIONS, ELECTRICAL INSTALLATIONS ASSOCIATED WITH FIRE DETECTORS AND ANNUNCIATORS. NO NONCOMPLIANCES WERE IDENTIFIED.

REVIEWED BY OMPA LOEB: ________ **DATE:** 12/10/80

REVIEWED BY NRR DL: DC Di ________ **DATE:** 12/10/80

THREE MILE ISLAND 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-289
2. Utility Contact: J. G. Mitchell (215) 921-6579
3. Licensed Thermal Power (MWt): 2535
4. Nameplate Rating (Gross MWe): 968 x 0.9 = 871
5. Design Electrical Rating (Net MWe): 819
6. Maximum Dependable Capacity (Gross MWe): 840
7. Maximum Dependable Capacity (Net MWe): 776
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	745.0	7,323.0	54,049.0
12. Hours Reactor Was Critical	.0	.0	31,731.8
13. Reactor Reserve Shutdown Hours	.0	.0	838.5
14. Hours Generator On-Line	.0	.0	31,180.9
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	0	76,531,071
17. Gross Electrical Energy (MWH)	0	0	25,484,330
18. Net Electrical Energy (MWH)	0	0	23,840,053
19. Unit Service Factor	.0	.0	57.7
20. Unit Availability Factor	.0	.0	57.7
21. Unit Capacity Factor (MDC Net)	.0	.0	56.8
22. Unit Capacity Factor (DER Net)	.0	.0	53.9
23. Unit Forced Outage Rate	100.0	100.0	23.1

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: INDEFINITE

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
1	02/17/79	F	745.0	D	4				REGULATORY RESTRAINT ORDER CONTINUES.

SUMMARY

THREE MILE ISLAND 1 REMAINS SHUTDOWN FOLLOWING THE ACCIDENT AT THREE MILE ISLAND 2.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

THREE MILE ISLAND 2

NRC POWER RESTRICTIONS: SEE "LICENSE STATUS"

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Pennsylvania/Dauphin 10 miles SE of Harrisburg
2.	Electric Reliability Council	Mid-Atlantic Area Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2772
5.	Design Electrical Rating (MWE Net)	906
6.	Date of Initial Criticality	3/28/78
7.	Date of Commercial Operation	12/30/78
8.	Condenser Cooling Method	CLOSED
9.	Condenser Cooling Water Source	Susquehanna River

Utility & Contractor Information

10.	Licensee	Metropolitan Edison Company
11.	Corporate Address	P. O. Box 542 Reading, PA 19603
12.	Corporate Contact	R. C. Arnold, Sr. Vice-President 100 Interpace Parkway Parsippany, NJ 07054
13.	Architect/Engineer	Burns & Roe
14.	Nuclear Steam Supply System	B&W
15.	Constructor	UE&C

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Section Chief	A. N. Fasano FTS: 590-1123 (717) 948-1123
19.	IE Resident Inspector	R. Conte FTS: 590-1125 (717) 948-1123
20.	TMI Program Director	B. Snyder
21.	Docket Number	50-320
22.	License Number/Date of Issue	DPR-73/2; 2/8/78
23.	Public Document Room	Government Publications Section State Library of Pennsylvania Education Building Commonwealth & Walnut Streets Harrisburg, Pennsylvania 17126

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO ORDER OF 2/11/80 (ATT 1- PROPOSED TS 6.8.1.8.G) PROCEDURES ASSOCIATED WITH THE REACTOR BUILDING PURGE WERE NOT PROPERLY IMPLEMENTED.
(8011 INFRACTION)

CONTRARY TO ORDER OF 2/11/80 (ATT 1 - PROP. TS 6.8.1.A) ON JUNE 29, 1980, THE LICENSEE WORKED ON THE MODIFIED HYDROGEN CONTROL SYSTEM EXHAUST FAN WHILE IT WAS EXHAUSTING THE REACTOR BUILDING ATMOSPHERE AND NO "WORK REQUEST" FORM WAS COMPLETED. DURING THE EVOLUTION THE INDIVIDUAL CAUSED THE FAN TO TRIP (AUTOMATIC SHUTDOWN).
(8011 INFRACTION)

CONTRARY TO ORDER OF 2/11/80 (ATT 1 - PROP. TS 6.8.2) AS OF JULY 4, 1980, A RECOVERY MODE PROCEDURE CHANGE (REVISION 1 TO OP 2104-4.82) REACTOR BUILDING ATMOSPHERE CLEANUP USING THE MODIFIED HYDROGEN CONTROL SYSTEM AND THE "B" TRAIN OF THE MODIFIED REACTOR BUILDING PURGE SYSTEM) WAS IMPLEMENTED FOR APPROXIMATELY 12 HOURS WITHOUT BEING SUBMITTED TO THE NRC PRIOR TO IMPLEMENTATION. THEREFORE, THIS REVISION WAS NOT SUBJECT TO APPROVAL BY THE NRC PRIOR TO IMPLEMENTATION.
(8011 DEFICIENCY)

CONTRARY TO THE TS 6.8.1.F AND HP 1682 PARAGRAPH 5.1.1 SEVERAL ITEMS WHICH HAD FIXED OR REMOVABLE CONTAMINATION IN EXCESS OF THE LIMITS OF HEALTH PHYSICS PROCEDURE 1682 WERE GIVEN UNCONDITIONAL RELEASE TO CLEAN AREAS FROM A UNIT 2 CONTROL POINT. MAXIMUM REMOVABLE CONTAMINATION EXCEEDED 10,000 DPM/100 CM2 AND MAXIMUM RADIATION LEVEL WAS 10 MRAD/HR ON ONE PIECE OF PIPE.
(8011 INFRACTION)

CONTRARY TO 10 CFR 19.11A(4) AS OF JUNE 30, 1980, NONE OF THE ORDERS ISSUED PURSUANT TO SUBPART B OF PART 2 SINCE JULY 20, 1979, WERE POSTED. IN ADDITION, THE NOTICES OF VIOLATION ISSUED BY THE OFFICE OF INSPECTION AND ENFORCEMENT (REGION I) DATED MAY 28, 1980, AND ONE BY THE OFFICE OF INSPECTION AND ENFORCEMENT DATED OCTOBER 25, 1979, WERE NOT POSTED IN A SUFFICIENT NUMBER OF PLACES. THAT IS, ONE (DATED MAY 28, 1980) WAS POSTED ONLY ON A BULLETIN BOARD IN UNIT 2 CONTROL ROOM AND THE OTHER (DATED OCTOBER 25, 1979) WAS NOT POSTED. THESE NOTICES OF VIOLATION INVOLVE CERTAIN UNCORRECTED RADIOLOGICAL WORKING CONDITIONS AND THIS BULLETIN BOARD IS NOT A NORMAL ACCESS TO RADIOLOGICAL AREAS SUCH AS THE AUXILIARY BUILDING.
(8011 DEFICIENCY)

LICENSE STATUS

By Order for Modification of License dated July 20, 1979, the licensee's authority to operate the facility was suspended and the licensee's authority was limited to maintenance of the facility in the present shutdown cooling mode (44 F.R. 45271).

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE			

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEM AND COMPONENT PROBLEMS:

CLEANUP OPERATIONS FROM 3/28/79 ACCIDENT.

PLANT STATUS:

DECAY HEAT IS BEING REMOVED BY CYCLIC NATURAL CIRCULATION THROUGH THE "A" ONCE THROUGH STEAM GENERATOR AND TO THE REACTOR BUILDING AMBIENT FROM BOTH RCS LOOPS.

+ LAST IE SITE INSPECTION DATE: 10/19/80

+ INSPECTION REPORT NO: 50-320/80-17

SUMMARY

INSPECTION SUMMARY

+ 50-320/80-04 - MAR 5-7 & 11-14: UNANNOUNCED INVESTIGATION OF AN ALLEGATION FROM A TMI EMPLOYEE WHO QUESTIONED THE RECENT REHIRING OF A FORMER EMPLOYEE. IT WAS ALLEGED THAT THE REHIRED EMPLOYEE MADE AN UNAUTHORIZED ENTRY ONTO THREE MILE ISLAND ON JULY 13, 1979 AT 11:10 P.M., ENTERED THE PUMPHOUSE, AND TRIPPED A NUCLEAR RIVER PUMP. THE INVESTIGATION DID NOT SUBSTANTIATE THE ALLEGED ACTIONS ASCRIBED TO THE FORMER EMPLOYEE. HOWEVER, THE UNEXPLAINED TRIP OF THE NUCLEAR RIVER PUMP (NRP-1A) ON JULY 13, 1979 AT 11:10 P.M. WAS VERIFIED. IN ADDITION, THREE INDIVIDUALS STATED THAT THE FORMER EMPLOYEE ADMITTED TO THEM, HIS ACTIONS WHICH WERE SIMILAR TO THE EVENT ALLEGED. THE FORMER EMPLOYEE STATED HE HAD FABRICATED THE STORY AS ALLEGED, BUT DENIED THAT HE HAD COMMITTED THE ACT.

+ 50-320/80-11 - JUN 1 - JUL 5: ROUTINE, ONSITE AND BACKSHIFT INSPECTION BY RESIDENT AND REGION-BASED INSPECTORS (218 HRS) OF SELECTED NEW AND REVISED PROCEDURES SUBMITTED FOR APPROVAL; PROCEDURE IMPLEMENTATION; POSTING OF NOTICES TO WORKERS; SHIFT MANNING/FIRE BRIGADE COMPOSITION; OFFSHIFT OPERATIONS ACTIVITIES: LICENSEE MEASURED REACTOR BUILDING CONCENTRATION UTILIZING NRC MOBILE LABORATORY; TRAINING PROGRAM FOR REACTOR BUILDING PURGE EVOLUTIONS; PWST CONSTRUCTION ACTIVITIES; SOLID WASTE STAGING CONSTRUCTION ACTIVITIES; SURVEILLANCE PROCEDURES INCLUDING ASSOCIATED DATA; AND RADIOACTIVE MATERIAL CONTROL. FIVE NONCOMPLIANCES WERE IDENTIFIED: FAILURE TO PROPERLY IMPLEMENT THE REACTOR BUILDING PURGE PROCEDURES; FAILURE TO UTILIZE THE WORK REQUEST SYSTEM TO CONDUCT SAFETY RELATED MAINTENANCE; FAILURE TO ASSURE CONTAMINATED MATERIAL IS NOT RELEASED TO UNCONTAMINATED AREAS; FAILURE TO SUBMIT A PROCEDURE REVISION TO THE NRC FOR APPROVAL; FAILURE TO PROPERLY POST NOTICES TO WORKERS.

REVIEWED BY OMPA LOEB: ______ **DATE:** 12/10/80

REVIEWED BY NRR DL: ______ **DATE:** 12/10/80

best copy available

THREE MILE ISLAND 2

AVERAGE DAILY POWER LEVEL (MWe)

OPERATING STATUS

1. Docket Number: 50-320
2. Utility Data Prepared By: R. A. Lengel (215) 921-6581
3. Licensed Thermal Power (MWt): 2772
4. Nameplate Rating (Gross MWe): 1068 X 0.9 = 961
5. Design Electrical Rating (Net MWe): 906
6. Maximum Dependable Capacity (Gross MWe): 0
7. Maximum Dependable Capacity (Net MWe): 0
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons

None

9. Power Level To Which Restricted, If Any (Net MWe): None
10. Reasons For Restrictions, If Any:

This Month Yr.-t

11. Hours In Reporting Period
12. Number Of Hours Reactor Was Critical
13. Reactor Reserve Shutdown Hours
14. Hours Generator On-Line
15. Unit Reserve Sh
16. Gro

(MWH)

1

20. U Availability Factor
21. Unit Capacity Factor (Using MDC Net) 1/
22. Unit Capacity Factor (Using DER Net) 1/
23. Unit Forced Outage Rate
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

None

25. If Shut Down At End Of Report Period, Estimated Date of Startup: NO ESTIMATE

NO DECISION ON FUTURE OPERATION
SHUTDOWN INDEFINITELY

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
SEE SUMMARY BELOW.									

SUMMARY

THE UNIT HAS REMAINED SHUTDOWN SINCE THE 03/28/79 ACCIDENT, WHICH CAUSED EXTENSIVE CORE DAMAGE.

AUTHORITY TO OPERATE FACILITY SUSPENDED BY NRC ORDER DATED JULY 20, 1979, WHICH REQUIRES LICENSEE TO MAINTAIN FACILITY IN A SHUTDOWN CONDITION.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

TROJAN

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Oregon/Columbia 42 miles N of Portland
2.	Electric Reliability Council	Western Systems Coordinating Council
3.	Type of Reactor	PWR
4.	Licensee Power Level (MWT)	3411
5.	Design Electrical Rating (MWE Net)	1130
6.	Date of Initial Criticality	December 15, 1975
7.	Date of Commercial Operation	May 20, 1976
8.	Condenser Cooling Method	Natural draft cooling tower
9.	Condenser Cooling Water Source	Columbia River

Utility & Contractor Information

10.	Licensee	Portland General Electric Co.
11.	Corporate Address	121 S. W. Salmon St. Portland, Oregon 97204
12.	Corporate Contact	C. Goodwin, Jr., Asst. V.P.
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Multiple

Regulatory Information

16.	IE Region responsible	V
17.	IE Branch Chief	J. L. Crews
18.	IE Section Chief	D. M. Sternberg
19.	IE Resident Inspector	M. H. Malmros (503) 556-3751
20.	Licensing Project Manager	C. M. Trammell
21.	Docket Number	50-344
22.	License Number/Date of Issue	NPF-1, 11/21/75
23.	Public Document Room	Columbia County Courthouse Law Library Circuit Court Room St. Helens, Oregon 97501

INSPECTION STATUS

ENFORCEMENT STATUS

INSPECTION STATUS

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

PURSUANT TO THE ASLB DECISION OF JULY 11, 1980, A LICENSE AMENDMENT WAS ISSUED ON JULY 25, 1980 AUTHORIZING THE LICENSEE TO MODIFY THE CONTROL BUILDING TO RESTORE THE INITIAL DESIGN MARGINS.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS

THE PLANT BEGAN AND ENDED THE MONTH AT FULL POWER. NO SHUTDOWNS OCCURRED THIS MONTH.

LAST IE SITE INSPECTION DATE: 10/01-31/80+

INSPECTION REPORT NO: 50-344/80-26+

SUMMARY

INSPECTION SUMMARY:

+ INSPECTION ON JULY 7-18, 1980 (REPORT NO. 50-344/80-16) AREAS INSPECTED: SPECIAL, ANNOUNCED APPRAISAL OF THE HEALTH PHYSICS PROGRAM, INCLUDING ORGANIZATION AND MANAGEMENT; QUALIFICATIONS AND TRAINING; QUALITY ASSURANCE; PROCEDURES; EXTERNAL AND INTERNAL EXPOSURE CONTROLS; SURVEY AND ACCESS CONTROLS; INSTRUMENTATION; ALARA; RADIOACTIVE WASTE; FACILITIES AND EQUIPMENT; AND ACCIDENT RESPONSE CAPABILITIES. THE INSPECTION INVOLVED 335 INSPECTOR HOURS ONSITE BY FOUR NRC INSPECTORS.

RESULTS: SIGNIFICANT STRENGTHS IN THE AREAS OF MANAGEMENT'S COMMITMENT TO AND SUPPORT OF RADIATION PROTECTION AND ALARA AND STAFFING WERE OBSERVED. SEVERAL SIGNIFICANT WEAKNESSES IN THE HEALTH PHYSICS PROGRAM WERE IDENTIFIED. THESE WEAKNESSES ARE IN THE AREAS OF TRAINING; SURVEILLANCE; RADIOACTIVE WASTE MANAGEMENT; ALARA; FACILITIES AND EQUIPMENT; AND EMERGENCY RESPONSE CAPABILITIES. TWO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED (INFRACTION - FAILURE TO PERFORM A REVIEW PURSUANT TO 10 CFR 50.59 AND INFRACTION - INSTRUCTION OF WORKERS PURSUANT TO 10 CFR 19.12.

+ INSPECTION ON OCTOBER 20-24, 1980 (REPORT NO. 50-344/80-23) REPORT BEING PREPARED; TO BE REPORTED NEXT MONTH.

+ INSPECTION ON OCTOBER 20-23, 1980 (REPORT NO. 50-344/80-25) AREAS INSPECTED: ROUTINE, UNANNOUNCED INSPECTION OF MAINTENANCE; PROCEDURES AND RESULTS; SURVEILLANCE PROCEDURES AND RESULTS; DESIGN, DESIGN CHANGES, AND MODIFICATIONS; ORGANIZATION AND ADMINISTRATION; RESPONSE TO IE BULLETINS/CIRCULARS; AND FOLLOWUP ON PREVIOUS INSPECTION ITEMS. ALSO INCLUDED INDEPENDENT INSPECTION EFFORT. THE INSPECTION INVOLVED 40 INSPECTOR HOURS ONSITE BY TWO NRC INSPECTORS.

RESULTS: NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

+ INSPECTION ON OCTOBER 1-31, 1980 (REPORT NO. 50-344/80-26) REPORT BEING PREPARED; TO BE REPORTED NEXT MONTH.

+ INSPECTION ON OCTOBER 27-31, 1980 (REPORT NO. 50-344/80-27) REPORT BEING PREPARED; TO BE REPORTED NEXT MONTH.

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-18/ 03L-0	09-03-80	10-02-80	FAILURE OF DIESEL AUXILIARY FEED PUMP TO START DURING TEST.
80-19/ 03L-0	09-24-80	10-14-80	RX TRIP SETPOINTS FOR HI STEAM FLOW WERE WRONG.
80-20/ 01T-0	10-03-80	10-14-80	FAILURE OF AFW PUMPS TO START ON LO LO SG W/L.

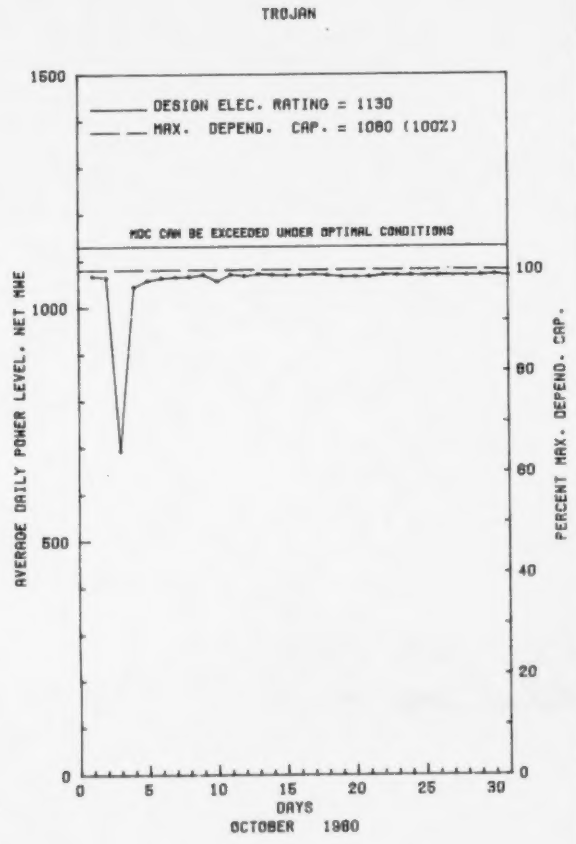
REVIEWED BY OMPA LOEB: P E Robe **DATE:** 12/10/80

REVIEWED BY NRR DL: CM Trammell **DATE:** 12/10/80

GPO 875-687

TROJAN

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-344
2. Utility Contact: G. G. Bair (503) 556-3713 x234
3. Licensed Thermal Power (MWt): 3411
4. Nameplate Rating (Gross MWe): 1280 x 0.95 = 1216
5. Design Electrical Rating (Net MWe): 1130
6. Maximum Dependable Capacity (Gross MWe): 1122
7. Maximum Dependable Capacity (Net MWe): 1080
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	745.0	7,320.0	36,528.0
12. Hours Reactor Was Critical	741.9	4,999.3	21,297.3
13. Reactor Reserve Shutdown Hours	.0	.0	2,171.8
14. Hours Generator On-Line	738.5	4,981.8	20,519.6
15. Unit Reserve Shutdown Hours	.0	.0	1,508.7
16. Gross Thermal Energy (MWH)	2,512,341	14,949,879	63,844,689
17. Gross Electrical Energy (MWH)	822,286	4,841,111	20,826,526
18. Net Electrical Energy (MWH)	785,365	4,597,401	19,630,720
19. Unit Service Factor	99.1	67.9	56.2
20. Unit Availability Factor	99.1	67.8	60.3
21. Unit Capacity Factor (MDC Net)	97.6	58.2	49.8
22. Unit Capacity Factor (DER Net)	93.3	55.6	47.6
23. Unit Forced Outage Rate	.9	12.1	26.9

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
REFUELING - APRIL 1981 - 45 DAYS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-09	10/03/80	F	6.5	G	3				LOSS OF MAIN GENERATOR FIELD OCCURRED WHEN PERSONNEL INADVERTENTLY REMOVED INPUT LEAD WHILE SETTING UP PLANT FOR POWER SYSTEMS STABLIZER TEST. GENERATOR TRIP RESULTED IN TURBINE AND REACTOR TRIP.
	10/03/80	F	.0	A	9	80-23/03L	CH	VALVEX	MAIN FEEDWATER REGULATING VALVE B FAILED TO CLOSE AUTOMATICALLY FOLLOWING PLANT TRIP BECAUSE A MANUAL VENT THROTTLE VALVE DID NOT ALLOW ENOUGH AIR VENTING. MANUAL VALVE WAS RESET AND IS BEING STUDIED FOR FOREIGN MATERIAL BLOCKAGE OR MALPOSITIONING.
	10/03/80	F	.0	G	9	80-20/01T	IF	PUMPXX	THE AUXILIARY FEEDWATER PUMPS FAILED TO START AUTOMATICALLY FROM LO-LO STEAM GENERATOR LEVEL FOLLOWING PLANT TRIP. INVESTIGATION SHOWED THAT THE LEADS TO A SLAVE RELAY HAD BEEN CONNECTED TO THE WRONG TERMINALS.

SUMMARY

TROJAN OPERATED AT OR NEAR FULL POWER FOR OCTOBER, WITH OUTAGES ON 10/03/80 DUE TO MULTIPLE REASONS LISTED IN DETAIL ABOVE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

TURKEY POINT 3

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Florida/Dade 25 miles S of Miami
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2200
5.	Design Electrical Rating (MWE Net)	693
6.	Date of Initial Criticality	October 20, 1972
7.	Date of Commercial Operation	December 14, 1972
8.	Condenser Cooling Method	Recirculating Canal System
9.	Condenser Cooling Water Source	Canal System

Utility & Contractor Information

10.	Licensee	Florida Power and Light Company
11.	Corporate Address	9250 West Flagler Street P. O. Box 013100 Miami, Florida 33101
12.	Corporate Contact	R. E. Uhrig, V.P. Nuclear Affairs
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. Lewis
18.	IE Section Chief	R. D. Martin
19.	IE Resident Inspector	A. J. Ignatonis (305) 245-7669
20.	Licensing Project Manager	M. Grotenhuis (301) 492-7128
21.	Docket Number	50-250
22.	License Number/Date of Issue	DPR-31, 7/19/72
23.	Public Document Room	Environmental and Urban Affairs Library Florida International University Miami, Florida 33199

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

SHUTDOWN FOR MAINTENANCE AND INSPECTION.

♦ LAST IE SITE INSPECTION DATE: OCTOBER 1-30, 1980

♦ INSPECTION REPORT NO: 50-250/80-33

SUMMARY

INSPECTION SUMMARY

♦ INSPECTION SEPTEMBER 9-12 (80-27): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 12 INSPECTOR-HOURS ON SITE IN THE AREAS OF IE BULLETIN 79-14 "SEISMIC ANALYSIS FOR AS-BUILT SAFETY RELATED PIPING SYSTEMS," CONTAINMENT PRESTRESSING SYSTEM SURVEILLANCE PROGRAM AND LICENSEE ACTION ON PREVIOUS INSPECTION FINDINGS. NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION AUGUST 27-29 (80-28): THIS SPECIAL, UNANNOUNCED MANAGEMENT INSPECTION INVOLVED 5 INSPECTOR-HOURS AT THE GENERAL OFFICES AND ONSITE IN THE AREA OF FOLLOWUP OF NONCOMPLIANCE RELATIVE TO 10 CFR PART 21 PROCEDURES. NONCOMPLIANCE 250/79-35-01, "10 CFR PART 21 PROCEDURES" ARE CLOSED.

REPORTS RECEIVED FROM LICENSEE

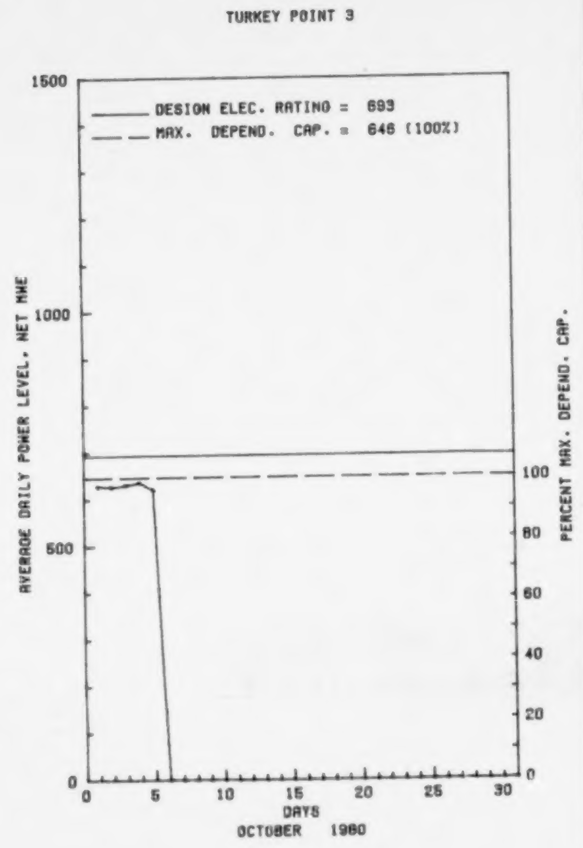
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE			

REVIEWED BY OMPA LOEB: DATE: 12/10/80

REVIEWED BY NRR DL: DATE: 12/10/80

TURKEY POINT 3

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-250
2. Utility Contact: V. T. Chilson (305) 552-3824
3. Licensed Thermal Power (MWt): 2200
4. Nameplate Rating (Gross MWe): 894 x 0.85 = 760
5. Design Electrical Rating (Net MWe): 693
6. Maximum Dependable Capacity (Gross MWe): 680
7. Maximum Dependable Capacity (Net MWe): 646

ITEM 21 CALCULATED BY USING A WEIGHTED AVERAGE

8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	745.0	7,320.0	69,321.6
12. Hours Reactor Was Critical	122.0	5,821.0	53,090.4
13. Reactor Reserve Shutdown Hours	.0	.0	213.3
14. Hours Generator On-Line	121.3	5,658.7	51,338.9
15. Unit Reserve Shutdown Hours	.0	.0	121.8
16. Gross Thermal Energy (MWH)	257,027	12,225,989	103,694,623
17. Gross Electrical Energy (MWH)	79,650	3,844,350	32,925,350
18. Net Electrical Energy (MWH)	72,844	3,641,796	31,163,850
19. Unit Service Factor	16.3	77.2	74.1
20. Unit Availability Factor	16.3	77.2	74.2
*21. Unit Capacity Factor (MDC Net)	15.1	77.0	69.9
22. Unit Capacity Factor (DER Net)	14.1	71.8	64.9
23. Unit Forced Outage Rate	.0	1.2	2.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
REFUEL, MAINT. & INSPECTIONS - MARCH 1, 1981 - MAY 10, 1981.
25. If Shut Down At End of Report Period, Estimated Date of Startup: 11/01/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
19	10/06/80	S	623.7	B	1		HB	HTEXCH	UNIT WAS REMOVED FROM SERVICE FOR SCHEDULED STEAM GENERATOR TUBE INSPECTION PROGRAM, MAINTENANCE, AND INSPECTIONS.

SUMMARY

TURKEY POINT 3 REMAINED ON-LINE THRU OCTOBER 5TH, AND COMMENCED A MAINTENANCE AND INSPECTION OUTAGE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

TURKEY POINT 4

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1. Location (state/county) (nearest population center)	Florida/Dade 25 miles S of Miami
2. Electric Reliability Council	Southeastern Electric Reliability Council
3. Type of Reactor	PWR
4. Power Level (MWT)	2200
5. Design Electrical Rating (MWE Net)	693
6. Date of Initial Criticality	June 11, 1973
7. Date of Commercial Operation	September 7, 1973
8. Condenser Cooling Method	Recirculating Canal System
9. Condenser Cooling Water Source	Canal System

Utility & Contractor Information

10. Licensee	Florida Power and Light Company
11. Corporate Address	9250 West Flagler Street P. O. Box 013100 Miami, Florida 33101
12. Corporate Contact	R. E. Uhrig, V. P. Nuclear Affairs
13. Architect/Engineer	Bechtel
14. Nuclear Steam Supply System	Westinghouse
15. Constructor	Bechtel

Regulatory Information

16. IE Region responsible	II
17. IE Branch Chief	R. C. Lewis
18. IE Section Chief	R. D. Martin
19. IE Resident Inspector	A. J. IGNATONIS (305) 245-7669
20. Licensing Project Manager	M. Grotenhuis (301) 492-7128
21. Docket Number	50-251
22. License Number/Date of Issue	DPR-41, 4/10/73
23. Public Document Room	Environmental and Urban Affairs Library Florida International University Miami, Florida 33199

INSPECTION STATUS

ENFORCEMENT STATUS

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

- ROUTINE OPERATION.
- LAST IE SITE INSPECTION DATE: OCTOBER 1-30, 1980
- INSPECTION REPORT NO: 50-251/80-32

SUMMARY

INSPECTION SUMMARY

- INSPECTION SEPTEMBER 9-12 (80-26): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 12 INSPECTOR-HOURS ON SITE IN THE AREAS OF IE BULLETIN 79-14 "SEISMIC ANALYSIS FOR AS-BUILT SAFETY RELATED PIPING SYSTEMS." CONTAINMENT PRESTRESSING SYSTEM SURVEILLANCE PROGRAM AND LICENSEE ACTION ON PREVIOUS INSPECTION FINDINGS. NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION AUGUST 27-29 (80-27): THIS SPECIAL, UNANNOUNCED MANAGEMENT INSPECTION INVOLVED 5 INSPECTOR-HOURS AT THE GENERAL OFFICES AND ONSITE IN THE AREA OF FOLLOWUP OF NONCOMPLIANCE RELATIVE TO 10 CFR PART 21 PROCEDURES. NONCOMPLIANCE 251/79-35-01, "10 CFR PART 21 PROCEDURES" IS CLOSED.

REPORTS RECEIVED FROM LICENSEE

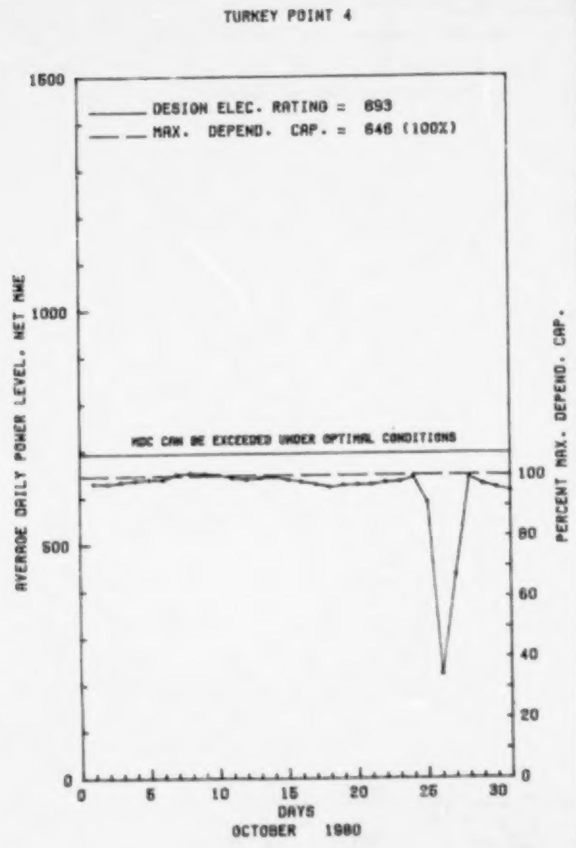
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE			

REVIEWED BY OMPA LOEB: PE Rohe **DATE:** 12/10/80

REVIEWED BY NRR DL: M Grotenhuis **DATE:** 12/10/80

TURKEY POINT 4

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-251
2. Utility Contact: V. T. Chilson (305) 552-3824
3. Licensed Thermal Power (MWt): 2200
4. Nameplate Rating (Gross MWe): 894 x 0.85 = 760
5. Design Electrical Rating (Net MWe): 693
6. Maximum Dependable Capacity (Gross MWe): 680
7. Maximum Dependable Capacity (Net MWe): 646
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE.
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

ITEM 21 CALCULATED BY USING A WEIGHTED AVERAGE.

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	745.0	7,320.0	63,049.0
12. Hours Reactor Was Critical	742.9	5,995.0	46,822.8
13. Reactor Reserve Shutdown Hours	.0	12.3	166.6
14. Hours Generator On-Line	741.7	5,906.5	45,089.0
15. Unit Reserve Shutdown Hours	.0	12.3	31.2
16. Gross Thermal Energy (MWH)	1,582,883	12,790,137	95,009,117
17. Gross Electrical Energy (MWH)	481,620	3,961,560	29,845,038
18. Net Electrical Energy (MWH)	456,455	3,755,903	28,284,899
19. Unit Service Factor	99.6	80.7	71.5
20. Unit Availability Factor	99.6	80.9	71.5
*21. Unit Capacity Factor (MDC Net)	94.8	79.4	69.7
22. Unit Capacity Factor (DER Net)	88.4	74.0	64.7
23. Unit Forced Outage Rate	.4	.3	2.9

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
REFUEL MAINT & INSPEC - NOVEMBER 9, 1980 - JANUARY 10, 1981.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
18	10/23/80	F	3.3	A	3		ED	GENERA	UNIT WAS TRIPPED BY THE REACTOR PROTECTION SYSTEM DUE TO THE LOSS OF POWER SUPPLY TO VITAL INSTRUMENT AC BUS 4AB.
19	10/26/80	F	.0	H	5		RC	ZZZZZZ	LOAD RESTRICTED TO 50% R.P. DUE TO AXIAL OFF-SET LIMITATIONS AFTER THE UNIT WAS RETURNED TO SERVICE. (10/26 & 10/27).

SUMMARY

TURKEY POINT 4 OPERATED AT NEAR FULL-POWER FOR OCTOBER, WITH ONE OUTAGE ON THE 25TH FOR EQUIPMENT FAILURE, AND ONE REDUCTION ON THE 26TH FOR REGULATORY RESTRICTION.

LEGEND

[1] Type:
F–Forced
S–Scheduled

[2] Reason:
A–Equipment Failure
B–Maintenance or Test
C–Refueling
D–Regulatory Restriction
E–Operator Training & License Examination
F–Administrative
G–Operational Error
H–Other

[3] Method:
1–Manual
2–Manual Scram
3–Automatic Scram
4–Continuations
5–Load Reduction
9–Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

VERMONT YANKEE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Vermont/Windham 5 miles S of Brattleboro
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	1593
5.	Design Electrical Rating (MWE Net)	514
6.	Date of Initial Criticality	March 24, 1972
7.	Date of Commercial Operation	November 30, 1972
8.	Condenser Cooling Method	Closed Cycle Cooling Towers
9.	Condenser Cooling Water Source	Connecticut River

Utility & Contractor Information

10.	Licensee	Vermont Yankee Nuclear Power Corp.
11.	Corporate Address	20 Turnpike Road Westboro, Massachusetts 01581
12.	Corporate Contact	D. W. Edwards Assistant to the Vice President
13.	Architect/Engineer	Ebasco
14.	Nuclear Steam Supply System	GE
15.	Constructor	Ebasco

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	IE Section Chief	T. T. MARTIN
19.	IE Resident Inspector	W. J. RAYMOND (802) 257-4310
20.	Licensing Project Manager	V. L. Rooney
21.	Docket Number	50-271
22.	License Number/Date of Issue	DPR-28, 3/21/72
23.	Public Document Room	Brooks Memorial Library 224 Main Street Brattleboro, Vermont 05301

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO T.S. 6.5.A VALVES V72-90E AND V72-152 WERE NOT ADDED TO INSTRUMENT AIR SYSTEM OPERATING PROCEDURE RP 2191 FOLLOWING INSTALLATION OF VALVES IN SYSTEM PER PDCR-79-08 DATED MARCH 23, 1979. VALVE LINEUP FILE FOR INSTRUMENT AIR SYSTEM DID NOT REFLECT SUBJECT VALVES FOR PERIOD OF JANUARY 10, 1980 TO AUGUST 10, 1980.
(8012 INFRACTION)

CONTRARY TO T.S. 6.6.A.2, ANSI N18.7-1976, AND AP0013 MAINTENANCE RECORDS OF SAFETY RELATED MAINTENANCE WERE NOT BEING MAINTAINED IN THAT WELD DATA SHEET FOR WELD IN SLC SYSTEM AND PACKING CHECK LIST FOR REPACKING A VALVE IN RHR SYSTEM COULD NOT BE LOCATED AND INSPECTIONS OF ROUTINE SAFETY RELATED MAINTENANCE WERE NOT DOCUMENTED.
(8012 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

- SYSTEM AND COMPONENT PROBLEMS:
- INSPECTIONS OF THE REACTOR WATER CLEANUP SYSTEM HAVE IDENTIFIED DEFECTS IN THE 4 INCH DIAMETER PIPING ON THE SUCTION SIDE OF THE RWCU PUMPS INSIDE CONTAINMENT. THE DEFECTS ARE LOCATED IN THE WELD PIPE FUSION LINE. LICENSEE INVESTIGATION IS CONTINUING.
- CORE SPRAY SPARGER EXAMINATION REVEALED A CRACK IN THE UPPER "C" JUNCTION BOX AT LOCATION 193 DEGREES. NO OTHER CRACKS HAVE BEEN IDENTIFIED. LICENSEE EVALUATION IN PROGRESS.
- ONE OF TWENTY JET PUMP HOLD DOWN ASSEMBLIES UT INSPECTED PER IEB 80-07 WAS FOUND TO HAVE CRACK INDICATIONS. THE HOLD-DOWN BEAM HAS BEEN REPLACED.
- DURING REFUELING OPERATIONS A NEW (UNEXPOSED) CONTROL ROD WAS DROPPED IN THE SPENT FUEL POOL WHEN A HOIST SWIVEL JOINT PARTED.
- DURING CONTROL ROD EXCHANGE A CONTROL ROD WAS DROPPED ONTO THE REACTOR CORE SINKING THREE IRRADIATED FUEL ASSEMBLIES. THERE WAS NO RELEASE OF RADIOACTIVITY. LICENSEE EVALUATION IN PROGRESS.
- WHILE REPLACING A LOCAL POINT RANGE MONITOR (LPRM) IT DISENGAGED FROM THE STRONGBACK USED TO POSITION AND SUPPORT THE LPRM AND IT FELL ONTO THE CORE. THE LPRM WAS RETRIEVED AND SUBSEQUENTLY INSPECTED WITH LICENSEE DETERMINATION OF NO DAMAGE.
- FACILITY ITEMS (PLANS AND PROCEDURES):
- REFUELING OUTAGE SCHEDULED TO LAST APPROXIMATELY EIGHT WEEKS.

PLANT STATUS:

- REFUELING OUTAGE COMMENCED SEPTEMBER 26, 1980.
- LAST IE SITE INSPECTION DATE: 10/20-31/80
- INSPECTION REPORT NO: 50-271/80-18

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-026/ 03L	09/03/80	10/03/80	LOSS OF CONTROL POWER TO HPCI PUMP DISCH VLV
80-027/ 03L	09/04/80	10/03/80	FAILURE OF RHR-SW PUMP TO MEET ISI D/P REQUIREMENTS DUE TO DEGRADATION OF PARALLEL PUMP DISCH CHECK VALVE
80-028/ 03L	09/08/80	10/07/80	LOSS OF ENVIRONMENTAL AIR COLLECTION STATION DUE TO BLOWN FUSE
80-029/ 03L	09/18/80	10/17/80	INCORRECT APRM GAIN SETTING DUE TO VARIED FUEL LENGTH

SUMMARY

INSPECTION SUMMARY

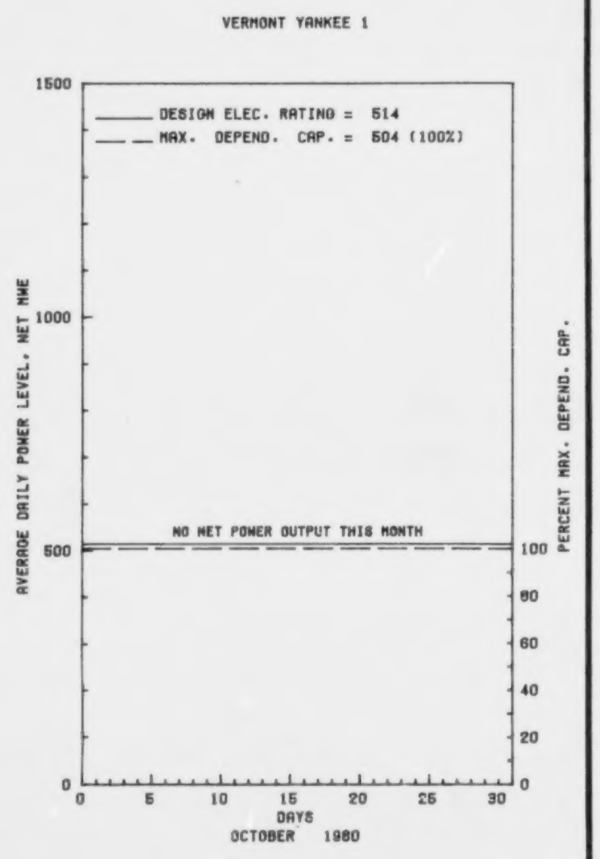
- 50-271/80-10 - JUL 5 - AUG 8: ROUTINE, UNANNOUNCED INSPECTION BY THREE RESIDENT INSPECTORS (162 HRS) OF PLANT OPERATIONS, INCLUDING REVIEW OF LOGS, RECORDS AND TOURS OF PLANT AREAS; SAFETY SYSTEM OPERATIONAL VERIFICATION; FOLLOWUP OF OPERATIONAL EVENTS; WITNESS OF SURVEILLANCE TESTS; FIRE PROTECTION PROGRAM SURVEILLANCES; TRANSPORTATION OF RADIOACTIVE MATERIAL; NEW FUEL RECEIPT INSPECTIONS; NEW FUEL INSPECTION AND CHANNELING ACTIVITIES; IEB 80-17, BWR SCRAM SYSTEM RESPONSES; INCLUDING SPECIAL TESTING, MEASUREMENTS; PROCEDURE CONTROLS AND TRANSIENT ANALYSES (TI 2515/39); BACKUP SCRAM VALVE OPERABILITY; DIESEL GENERATOR AIR START SYSTEM MODIFICATION PACKAGE; LOCATION OF ESSENTIAL LOAD CENTERS; LER 80-24 FOLLOWUP; PORC ACTIVITIES AND FUNCTIONS; LICENSEE PROBLEM REPORTS; LICENSEE PLANS FOR COPING WITH STRIKES; IMPLEMENTATION OF STRIKE PLANS; AND, OBSERVATIONS OF PLANT PHYSICAL SECURITY. NO NONCOMPLIANCES WERE IDENTIFIED.
- 50-271/80-12 - AUG 12-15: ROUTINE, UNANNOUNCED INSPECTION BY ONE REGION-BASED INSPECTOR (30 HRS) AND TWO RESIDENT INSPECTORS (14 HRS) ON PREVIOUS INSPECTION FINDINGS; ADMINISTRATIVE CONTROLS FOR SAFETY-RELATED MAINTENANCE; SAFETY-RELATED MAINTENANCE ACTIVITIES; MAINTENANCE PERSONNEL QUALIFICATIONS; AND A FACILITY TOUR. TWO NONCOMPLIANCES WERE IDENTIFIED: FAILURE TO UPDATE VALVE LINEUP PROCEDURES TO INCLUDE NEWLY INSTALLED INSTRUMENT AIR VALVES; FAILURE TO MAINTAIN DOCUMENTATION OF SAFETY RELATED MAINTENANCE AND INSPECTION ACTIVITIES.

REVIEWED BY OMPA LOEB: DATE: 12-10-80

REVIEWED BY NRR DL: DATE: 12/10/80

VERMONT YANKEE

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-271
2. Utility Contact: F.C. BEERS (617) 366-9011
3. Licensed Thermal Power (MWt): 1593
4. Nameplate Rating (Gross MWe): 626 x 0.9 = 563
5. Design Electrical Rating (Net MWe): 514
6. Maximum Dependable Capacity (Gross MWe): 535
7. Maximum Dependable Capacity (Net MWe): 504
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	745.0	7,320.0	71,114.8
12. Hours Reactor Was Critical	.0	6,266.6	57,425.2
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	.0	6,200.1	55,527.9
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	9,347,970	78,572,581
17. Gross Electrical Energy (MWH)	0	3,136,006	26,105,676
18. Net Electrical Energy (MWH)	0	2,972,800	24,741,173
19. Unit Service Factor	.0	84.7	78.1
20. Unit Availability Factor	.0	84.7	78.1
21. Unit Capacity Factor (MDC Net)	.0	80.6	69.0
22. Unit Capacity Factor (DER Net)	.0	79.0	67.7
23. Unit Forced Outage Rate	.0	1.9	6.0

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 12/01/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-21	09/26/80	S	745.0	C	4		RC	FUELXX	REFUELING OUTAGE CONTINUES.

SUMMARY

VERMONT YANKEE REMAINED OFF-LINE DURING OCTOBER FOR CONTINUATION OF REFUELING OUTAGE.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1-Manual
- 2-Manual Scram
- 3-Automatic Scram
- 4-Continuations
- 5-Load Reduction
- 9-Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

YANKEE ROWE

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Massachusetts/Franklin 25 miles NE of Pittsfield
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	600
5.	Design Electrical Rating (MWE Net)	175
6.	Date of Initial Criticality	August 19, 1960
7.	Date of Commercial Operation	July 1961
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Deerfield River

Utility & Contractor Information

10.	Licensee	Yankee Atomic Electric Company
11.	Corporate Address	20 Turnpike Road Westboro, Massachusetts 01581
12.	Corporate Contact	D. W. Edwards, Asst. to V.P.
13.	Architect/Engineer	Stone & Webster
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Section Chief	T. T. MARTIN
19.	IE Resident Inspector	T. Foley (413) 424-5242
20.	Licensing Project Manager	R. Caruso (X27232)
21.	Docket Number	50-29
22.	License Number/Date of Issue	DPR-3, 12/24/63
23.	Public Document Room	Greenfield Community College 1 College Drive Greenfield Massachusetts 01301

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

- SYSTEM AND COMPONENT PROBLEMS:
- ON OCTOBER 29, 1980 DURING A PLANT STARTUP. THE PLANT EXPERIENCED A THROTTLE VALVE FAILURE. REPAIRS WERE COMPLETED WITHIN 12 HOURS AND STARTUP WAS RECOMMENCED IMMEDIATELY AFTER.
- MANAGERIAL ITEMS:
- A FIVE MAN TEAM FROM NRR VISITED THE SITE ON NOVEMBER 3, 1980 TO REVIEW THE AUXILIARY FEEDWATER SYSTEM FOR SEISMIC QUALIFICATION. THE REVIEW IS PART OF AN ACCELERATED SEP REVIEW OF THE AUXILIARY FEEDWATER SYSTEM.

PLANT STATUS:

- ON OCTOBER 29, 1980 THE PLANT RESUMED A NORMAL STARTUP FROM COLD SHUTDOWN PROCEDURE AND ACHIEVED FULL POWER OPERATIONS ON NOVEMBER 5, 1980.
- LAST IE SITE INSPECTION DATE: 9/16 - 10/30/80
- INSPECTION REPORT NO: 50-29/80-16

SUMMARY

INSPECTION SUMMARY

- 50-29/80-14 - AUG 8 - SEP 15: ROUTINE UNANNOUNCED INSPECTION BY ONE RESIDENT INSPECTOR (32 HRS) OF PLANT OPERATIONS INCLUDING: LOG AND RECORD REVIEW; TOUR OF THE FACILITY; OBSERVATION OF PHYSICAL SECURITY; RADIATION PROTECTION CONTROLS; HOUSEKEEPING; FIRE PROTECTION; SURVEILLANCE TESTING; MAINTENANCE ACTIVITIES AND FOLLOWUP OF SELECTED IE BULLETINS AND CIRCULARS. ONE NONCOMPLIANCE WAS IDENTIFIED: FAILURE TO MAINTAIN A HIGH RADIATION AREA BARRICADED.
- 50-29/80-15 - SEP 7-12: PHYSICAL SECURITY INSPECTION BY ONE REGION-BASED INSPECTOR (42 HRS) OF: SECURITY PLAN; SECURITY PROGRAM AUDIT; SECURITY ORGANIZATION; PHYSICAL BARRIERS (PROTECTED AREAS); PHYSICAL BARRIERS (VITAL AREAS); LIGHTING; ACCESS CONTROL (IDENTIFICATION, AUTHORIZATION AND BADGING); ACCESS CONTROL (SEARCH); ACCESS CONTROL (ESCORTING); ACCESS CONTROL (VITAL AREAS); ALARM STATIONS; DETECTION AIDS; ASSESSMENT AIDS; COMMUNICATIONS; TESTING AND MAINTENANCE; COMPENSATORY MEASURES; POWER SUPPLY; RESPONSE; LOCKS, KEYS, AND COMBINATIONS; AND RECORDS AND REPORTS; FOLLOWUP ON A PREVIOUSLY IDENTIFIED ITEM OF NONCOMPLIANCE RELATED TO KEY CONTROL. NO NONCOMPLIANCES WERE IDENTIFIED.
- 50-29/80-17 - SEP 22-24: ROUTINE, UNANNOUNCED INSPECTION BY ONE REGION-BASED INSPECTOR (15 HRS) OF THE CONTAINMENT INTEGRATED LEAK RATE TEST (CILRT) AND LICENSEE ACTION ON PREVIOUS INSPECTION FINDINGS. NO NONCOMPLIANCES WERE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

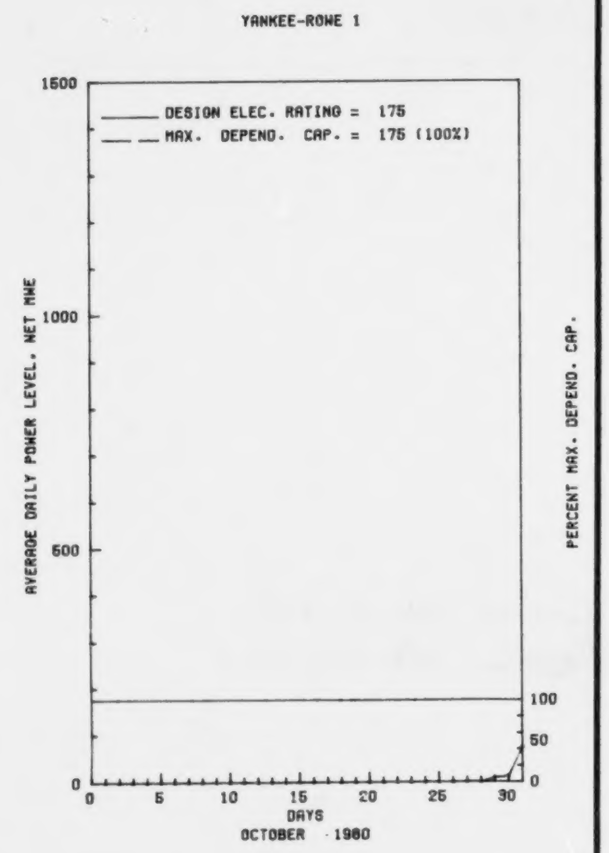
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-013/03L	06/30/80	07/30/80	S/G BLOWDOWN TRIP VALVE FAILED TYPE C TEST
80-014/03L	07/09/80	08/08/80	WELD DEFECTS IN STAGNANT BORATED LINES
80-015/03L	07/21/80	08/20/80	FIRE HYDRANTS REMOVED FROM SERVICE FOR REPAIRS
80-016/99L	07/31/80	09/09/80	FIRE PROTECTION SYSTEM OUT OF SERVICE FOR MODIFICATION

REVIEWED BY OMPA LOEB: ______________ **DATE:** 12/10/80

REVIEWED BY NRR DL: ______________ **DATE:** [illegible]

YANKEE ROWE

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-029
2. Utility Contact: F.C. BEERS (617) 366-9011
3. Licensed Thermal Power (MWt): 600
4. Nameplate Rating (Gross MWe): 185 x 1.0 = 185
5. Design Electrical Rating (Net MWe): 175
6. Maximum Dependable Capacity (Gross MWe): 180
7. Maximum Dependable Capacity (Net MWe): 175
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

CAPACITY FACTORS CALCULATED WITH A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	745.0	7,320.0	174,957.0
12. Hours Reactor Was Critical	65.8	911.6	138,876.2
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	40.2	479.9	134,507.3
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	11,610	265,379	72,013,701
17. Gross Electrical Energy (MWH)	2,681	83,316	22,155,469
18. Net Electrical Energy (MWH)	2,307	78,056	20,744,200
19. Unit Service Factor	5.4	6.6	76.9
20. Unit Availability Factor	5.4	6.6	76.9
21. Unit Capacity Factor (MDC Net)	1.8	6.1	70.0
22. Unit Capacity Factor (DER Net)	1.8	6.1	70.0
23. Unit Forced Outage Rate	94.6	91.9	5.4

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
1	01/19/80	F	686.8	A	4				THE PLANT WAS SHUTDOWN TO INVESTIGATE AND REPAIR THE TURBINE ROTOR DAMAGE.
2	10/29/80	F	24.8	B	3	80-20	CC	VALVEX	THE NORMAL TURBINE OVERSPEED TEST WAS CONDUCTED AFTER NINE (9) HOURS OF OPERATION. A BROKEN THROTTLE POPPET VALVE WAS FOUND AND REPAIRED PRIOR TO STARTUP.

SUMMARY

YANKEE-ROWE RETURNED ON-LINE ON THE 29TH AFTER EXTENSIVE ROTOR REPAIRS.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

ZION 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Illinois/Lake 40 miles N of Chicago
2.	Electric Reliability Council	Mid-America Interpool Network
3.	Type of Reactor	PWR
4.	Power Level (MWT)	3250
5.	Design Electrical Rating (MWE Net)	1040
6.	Date of Initial Criticality	June 19, 1973
7.	Date of Commercial Operation	December 31, 1973
8.	Condenser Cooling Method	Straight Through
9.	Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10.	Licensee	Commonwealth Edison Co.
11.	Corporate Address	P. O. Box 767 Chicago, Illinois
12.	Corporate Contact	J. S. Abel, Director of Nuclear Licensing
13.	Architect/Engineer	Sargent & Lundy
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Various

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	R. L. Spessard
19.	IE Resident Inspector	J. E. Kohler (312) 746-2313
20.	Licensing Project Manager	D. Wigginton
21.	Docket Number	50-295
22.	License Number/Date of Issue	DPR-39 issued 4/6/73
23.	Public Document Room	Zion-Benton Public Library 2600 Emmans Avenue Zion, Illinois 60099

INSPECTION STATUS

ENFORCEMENT STATUS

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

UNIT 1 OPERATING ROUTINELY.

+LAST IE SITE INSPECTION DATE: AUGUST 27-28 AND SEPTEMBER 16-18, 1980

+ INSPECTION REPORT NO: 80-19

SUMMARY

INSPECTION SUMMARY

+ INSPECTION ON AUGUST 1 THROUGH SEPTEMBER 30, (80-17): ROUTINE, UNANNOUNCED RESIDENT INSPECTION OF REACTOR TRIPS, SAFEGUARDS SYSTEMS ACTUATION, SVAG VALVE OPERATION, FUEL BUILDING CHARCOAL FILTER OPERATIONS, TECHNICAL SPECIFICATIONS ON RADIATION MONITORS, HIGH RADIATION COMPONENTS IN THE SPENT FUEL PIT, TESTING OF ESF RESET CONTROLS, LOSS OF POWER TO E.H.S., MODIFICATION TO W-2 SWITCHES, FAILURE TO PZR. SAMPLE VALVE, OPERATIONAL SAFETY VERIFICATION, MAINTENANCE ACTIVITIES, SURVEILLANCE ACTIVITIES AND LER AND BULLETIN FOLLOWUP. THE INSPECTION INVOLVED 321 INSPECTOR-HOURS ONSITE BY TWO NRC INSPECTORS INCLUDING 44 INSPECTOR-HOURS ONSITE DURING OFF-SHIFTS. OF THE AREAS INSPECTED, ONE ITEM OF NONCOMPLIANCE (VIOLATION OF H-2 PURGE FAN SURVEILLANCE REQUIREMENTS) WAS IDENTIFIED.

+ INSPECTION ON AUGUST 27-28 AND SEPTEMBER 16-18, (80-19): LICENSEE ACTIONS RELATIVE TO IE BULLETIN 79-14, INCLUDING GENERAL DISCUSSION ON NRC REQUIREMENTS, WORK PROCEDURE REVIEW, AND OBSERVATION OF WORK. THE INSPECTION INVOLVED 36 INSPECTOR-HOURS ONSITE BY ONE NRC INSPECTOR. NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

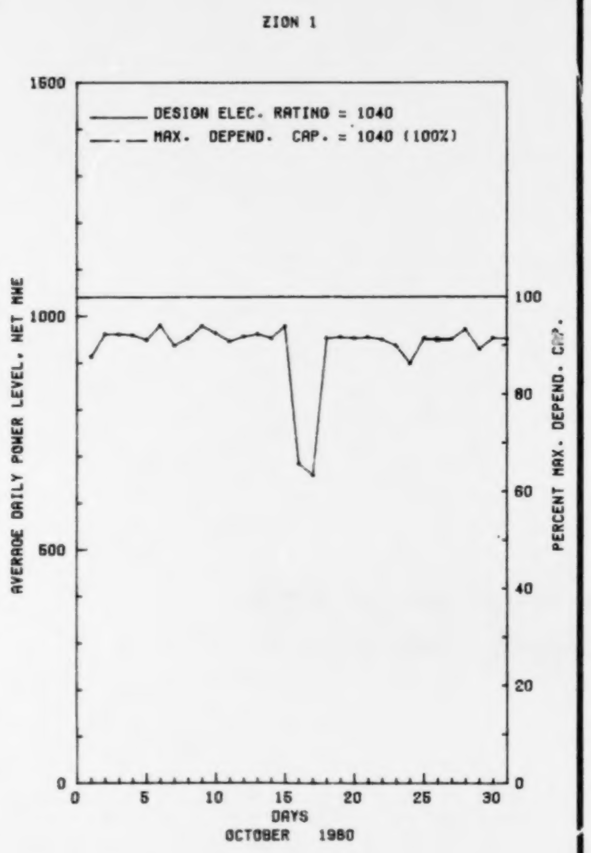
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-41/ 03L-0	08/29/80	10/06/80	RECALIBRATION OF OFFGAS RAD MONITOR WAS NOT COMPLETED WITHIN TS TIME LIMIT BECAUSE OF UNAVAILABILITY OF CAL. GAS.
80-42/ 03L-0	09/12/80	10/10/80	ERRATIC READINGS AND ALARMS INTERMITTENTLY RECEIVED FROM AUX. BLDG. - HVAC AREA RAD MONITOR.

REVIEWED BY OMPA LOEB: T. Cintula DATE: 18·10·80

REVIEWED BY NRR DL: D L Wigginton DATE: 12-10-80

ZION 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-295
2. Utility Contact: J. COOK (312) 746-2084
3. Licensed Thermal Power (MWt): 3250
4. Nameplate Rating (Gross MWe): 1220 x 0.9 = 1098
5. Design Electrical Rating (Net MWe): 1040
6. Maximum Dependable Capacity (Gross MWe): 1085
7. Maximum Dependable Capacity (Net MWe): 1040
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	745.0	7,320.0	59,928.0
12. Hours Reactor Was Critical	745.0	6,088.1	43,195.8
13. Reactor Reserve Shutdown Hours	.0	.0	2,621.8
14. Hours Generator On-Line	745.0	5,926.3	42,013.6
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	2,342,762	18,381,219	117,764,220
17. Gross Electrical Energy (MWH)	723,550	5,697,900	37,929,950
18. Net Electrical Energy (MWH)	695,442	5,431,396	35,915,642
19. Unit Service Factor	100.0	81.0	70.1
20. Unit Availability Factor	100.0	81.0	70.1
21. Unit Capacity Factor (MDC Net)	89.8	71.3	57.6
22. Unit Capacity Factor (DER Net)	89.8	71.3	57.6
23. Unit Forced Outage Rate	.0	19.0	11.3

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
REFUELING SCHEDULED FOR JAN. 16, 1981 - APPROX. 8 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
21	10/16/80	F	.0	B	5				POWER REDUCED FOR 1C FEEDWATER PUMP MAINTENANCE.

SUMMARY

ZION 1 OPERATED NORMALLY DURING OCTOBER WITH ONE REDUCTION ON THE 16TH FOR FEEDWATER PUMP MAINTENANCE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

ZION 2

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Illinois/Lake 40 miles N of Chicago
2.	Electric Reliability Council	Mid-America Interpool Network
3.	Type of Reactor	PWR
4.	Licenses Power Level (MWT)	3250
5.	Design Electrical Rating (MWE Net)	1040
6.	Date of Initial Criticality	December 24, 1973
7.	Date of Commercial Operation	September 17, 1974
8.	Condenser Cooling Method	Straight Through
9.	Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10.	Licensee	Commonwealth Edison Co.
11.	Corporate Address	P. O. Box 767 Chicago, Illinois
12.	Corporate Contact	J. S. Abel, Director of Nuclear Licensing
13.	Architect/Engineer	Sargent & Lundy
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Various

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	R. L. Spessard
19.	IE Resident Inspector	J. E. Kohler (312) 746-2313
20.	Licensing Project Manager	D. Wigginton
21.	Docket Number	50-304
22.	License Number/Date of Issue	DPR-48, 11/14/73
23.	Public Document Room	Zion-Benton Public Library 2600 Emmans Avenue Zion, Illinois 60099

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

UNIT 2 OPERATING ROUTINELY.

+LAST IE SITE INSPECTION DATE: AUGUST 27-28 AND SEPTEMBER 16-18, 1980

+ INSPECTION REPORT NO: 80-19

SUMMARY

INSPECTION SUMMARY

+ INSPECTION ON AUGUST 1 THROUGH SEPTEMBER 30, (80-17): ROUTINE, UNANNOUNCED RESIDENT INSPECTION OF REACTOR TRIPS, SAFEGUARDS SYSTEMS ACTUATION, SVAG VALVE OPERATION, FUEL BUILDING CHARCOAL FILTER OPERATIONS, TECHNICAL SPECIFICATIONS ON RADIATION MONITORS, UNIT 2 NIS CHANNEL, HIGH RADIATION COMPONENTS IN THE SPENT FUEL PIT, TESTING OF ESF RESET CONTROLS, LOSS OF POWER TO E.H.S., MODIFICATION TO W-2 SWITCHES, FAILURE TO PZR. SAMPLE VALVE, OPERATIONAL SAFETY VERIFICATION, MAINTENANCE ACTIVITIES, SURVEILLANCE ACTIVITIES AND LER AND BULLETIN FOLLOWUP. THE INSPECTION INVOLVED 321 INSPECTOR-HOURS ONSITE BY TWO NRC INSPECTORS INCLUDING 44 INSPECTOR-HOURS ONSITE DURING OFF-SHIFTS. OF THE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED.

+ INSPECTION ON AUGUST 27-28 AND SEPTEMBER 16-18, (80-19): LICENSEE ACTIONS RELATIVE TO IE BULLETIN 79-14, INCLUDING GENERAL DISCUSSION ON NRC REQUIREMENTS, WORK PROCEDURE REVIEW, AND OBSERVATION OF WORK. THE INSPECTION INVOLVED 36 INSPECTOR-HOURS ONSITE BY ONE NRC INSPECTOR. NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

NRC POWER RESTRICTIONS: NONE

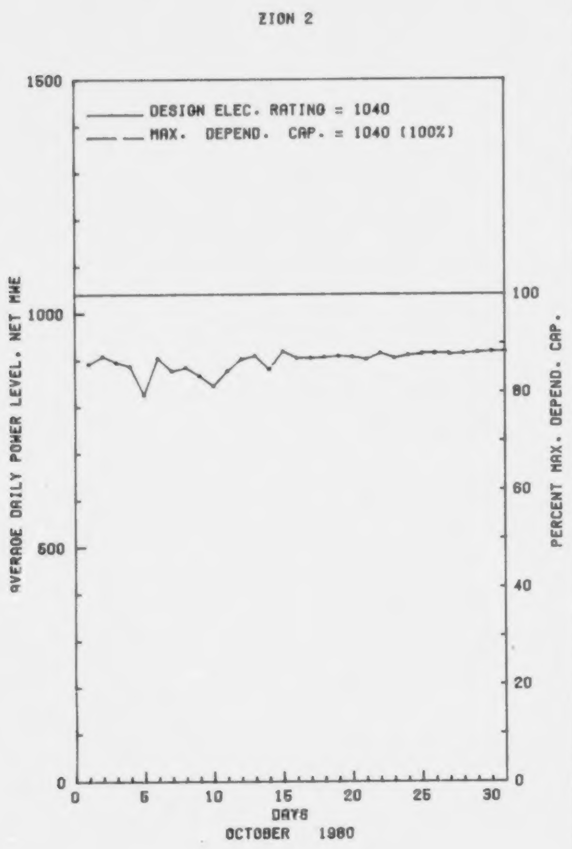
REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-24/ 99X	07/23/80	10/10/80	TRAIN B OF THE SI SYSTEM WAS INADVERTENTLY INITIATED DURING RETURN TO SERVICE.

REVIEWED BY OMPA LOEB: T. Cintula **DATE:** 12·10·80

REVIEWED BY NRR DL: DWigginton **DATE:** 12-10-80

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-304
2. Utility Contact: JOAN COOK (312) 746-2084
3. Licensed Thermal Power (MWt): 3250
4. Nameplate Rating (Gross MWe): 1220 x 0.9 = 1098
5. Design Electrical Rating (Net MWe): 1040
6. Maximum Dependable Capacity (Gross MWe): 1085
7. Maximum Dependable Capacity (Net MWe): 1040
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	745.0	7,320.0	53,641.0
12. Hours Reactor Was Critical	745.0	4,806.4	38,410.0
13. Reactor Reserve Shutdown Hours	.0	.0	226.1
14. Hours Generator On-Line	745.0	4,614.5	37,428.4
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	2,230,280	13,755,264	106,390,582
17. Gross Electrical Energy (MWH)	696,935	4,412,815	34,134,035
18. Net Electrical Energy (MWH)	668,862	4,184,392	32,385,850
19. Unit Service Factor	100.0	63.0	69.8
20. Unit Availability Factor	100.0	63.0	69.8
21. Unit Capacity Factor (MDC Net)	86.3	55.0	58.1
22. Unit Capacity Factor (DER Net)	86.3	55.0	58.1
23. Unit Forced Outage Rate	.0	16.6	15.8

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	NONE								

SUMMARY

ZION 2 OPERATED NORMALLY DURING OCTOBER WITH NO REDUCTIONS OR OUTAGES REPORTED.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

SECTION 3 – APPENDIX

STATUS OF SPENT FUEL STORAGE CAPABILITY

DATA AS OF 10/31/80

Facility	Core Size [a] (No. of Assemblies)	Present Authorized Storage Pool Capacity (Fuel Assemblies)	No. of Assemblies Stored	Remaining Capacity (No. of Assemblies)	Remaining Capacity If Pending Request Approved (No. of Assemblies)	Scheduled Date For Next Refueling	Will Fill Present [b] Authorized Capacity
I. Pressurized Water Reactors							
ARKANSAS 1	177	590	176	414		01-81	1986
ARKANSAS 2	177	486	58	428		03-81	1989
BEAVER VALLEY 1	157	833	0	833		N/S	1995
CALVERT CLIFFS 1	217	1056 (c)	456 (c)	692 (c)		10-80	1985
CALVERT CLIFFS 2	217					01-81	
COOK 1	193	500	193	307	1657	N/S	1985
COOK 2	193					N/S	
CRYSTAL RIVER 3	177	256	112	144	1109	N/S	1982
DAVIS-BESSE 1	177	735	52	683		03-82	1989
FARLEY 1	157	675	0	675		09-80	1990
FORT CALHOUN 1	133	483	197	286		03-81	1985
GINNA	121	595	156	439		N/S	1987
HADDAM NECK	157	1168	389	779		09-81	1994
INDIAN POINT 1	0	288	160	128		N/S	
INDIAN POINT 2	193	482	200	282		N/S	1984
INDIAN POINT 3	193	837	64	773		N/S	1989
KEWAUNEE	121	990	120	870		N/S	1999
MAINE YANKEE	217	953	433	520		N/S	1986
MILLSTONE 2	217	667	216	451		N/S	1987
NORTH ANNA 1	157	400	0	400	966	N/S	1984
NORTH ANNA 2 (d)	157	400	0	400			
OCONEE 1	177	1224 (l)	737	487 (l)	755	05-81	1980
OCONEE 2	177					06-81	
OCONEE 3	177					12-80	
PALISADES	204	798	273	525		N/S	1985
POINT BEACH 1	121	1502 (c)	276 (c)	1226 (c)		11-80	1995
POINT BEACH 2	121					03-81	
PRAIRIE ISLAND 1	121	687 (c)	280 (c)	407 (c)		09-80	1984
PRAIRIE ISLAND 2	121					01-81	
RANCHO SECO 1	177	579	164	415		04-81	1987
ROBINSON 2	157	272	106	166 (e)		N/S	1980 (g)
SALEM 1	193	264	104	224	1130	05-80	1982
SAN ONOFRE 1	157	216	94	119		N/S	1983
SEQUOYAH 1 (d)							
ST LUCIE 1	217	728	60	668		N/S	1990
SURRY 1	157	1044 (c)	436 (c)	608 (c)		09-80	1983
SURRY 2	157					05-80	
THREE MILE ISLAND 1	177	752	208	544		N/S	1986
THREE MILE ISLAND 2	177	442	0	442		N/S	1986
TROJAN	193	651	128	507		04-81	1988
TURKEY POINT 3	157	621	346 (c)	275 (c)		N/S	1987
TURKEY POINT 4	157					N/S	
YANKEE-ROWE 1	76	391	149	242	572	N/S	1985
ZION 1	193	2112 (c)	508 (c)	1604 (c)	1676	01-81	1993
ZION 2	193					N/S	

Facility	Core Size [a] (No. of Assemblies)	Present Authorized Storage Pool Capacity (Fuel Assemblies)	No. of Assemblies Stored	Remaining Capacity (No. of Assemblies)	Remaining Capacity If Pending Request Approved (No. of Assemblies)	Scheduled Date For Next Refueling	Will Fill Present [b] Authorized Capacity
II. Boiling Water Reactors							
BIG ROCK POINT 1	84	193	88	105	343	10-80	1983
BROWNS FERRY 1	764	3471	536	2795		04-81	1985
BROWNS FERRY 2	764	1080	640	440		09-80	1985
BROWNS FERRY 3	764	1528	516	1012		11-80	1985
BRUNSWICK 1	560	(f)	154PWR+476BWR			N/S	1986
BRUNSWICK 2	560		105PWR+264BWR			N/S	1986
COOPER STATION	548	2366	284	2082		N/S	1995
DRESDEN 1	464	672	221	451		N/S	1990
DRESDEN 2	724	2840 (c)	1069 (c)	1771 (c)	6491 (c)	12-80	1985
DRESDEN 3	724					N/S	
DUANE ARNOLD	368	2050	364	1686		04-81	1998
FITZPATRICK	560	760	428	492	1816	09-81	1981
HATCH 1	560	840	260	580		12-80	1988
HATCH 2	560	1120	0	1120		10-80	
HUMBOLDT BAY	172	487	251	236		N/S	
LA CROSSE	72	138	113	21	327	11-80	1990
MILLSTONE 1	580	2184	776	1408		09-80	1991
MONTICELLO	484	2237	828	1409		03-81	1991
NINE MILE POINT 1	532	1984	660	1324	2349	N/S	1990
OYSTER CREEK 1	560	1800	781	1020		N/S	1987
PEACH BOTTOM 2	764	2816	910	1906		01-82	1990
PEACH BOTTOM 3	764	2816	712	2104		03-81	1991
PILGRIM 1	580	2320	764	1556		09-81	1990
QUAD CITIES 1	724	1460	820	864		09-80	1985
QUAD CITIES 2	724	1460	672	968		N/S	1984
VERMONT YANKEE 1	368	2000	894	1106		09-80	1988
III. Independent Spent Fuel Storage Installations (h)							
Morris Operations		750 MTU (j)	315	385 MTU (j)	1490 MTU (j)		
NFS(i)		250 MTU	170 MTU	80 MTU			

(a) At each refueling outage approximately 1/3 of a PWR core and 1/4 of a BWR core is off-loaded.
(b) Some of these dates have been adjusted by staff assumptions.
(c) This is the total for both units.
(d) Plant not in commercial operation.
(e) Some spent fuel stored at Brunswick.
(f) Authorized a total 2088 BWR and 304 PWR assemblies for both pools.
(g) Robinson 2 assemblies being shipped to Brunswick for storage.
(h) Capacity is in metric tons of uranium; 1 MTU = 2 PWR assemblies or 5 BWR assemblies.
(i) No longer accepting spent fuel.
(j) Racked for 700 MTU.
(k) Reserved
(l) This is the station total.

N/S = Not Scheduled

REACTOR-YEARS OF EXPERIENCE

10/31/80

The total of Reactor years of experience is as the sum of all calendar days for each unit, from the date that electricity was first generated until a final shutdown date or the status date, whichever comes first, divided by 365.25 days/year.

If a date is unknown, the first day of the first month of operation is substituted. Units which have not yet generated electricity but which are licensed are listed but not included in the computation.

LICENSED OPERATING ELECTRICAL PRODUCING UNITS

UNIT NAME	FIRST ELEC. GENERATION	YEARS EXPERIENCE
ARKANSAS 1	08/01/74	6.25
ARKANSAS 2	12/26/78	1.85
BEAVER VALLEY 1	06/14/76	4.38
BIG ROCK POINT 1	12/08/62	17.90
BROWNS FERRY 1	10/15/73	7.05
BROWNS FERRY 2	08/28/74	6.18
BROWNS FERRY 3	09/12/76	4.14
BRUNSWICK 1	12/04/76	3.91
BRUNSWICK 2	04/29/75	5.51
CALVERT CLIFFS 1	01/03/75	5.83
CALVERT CLIFFS 2	12/07/76	3.90
COOK 1	02/10/75	5.72
COOK 2	03/22/78	2.61
COOPER STATION	05/10/74	6.48
CRYSTAL RIVER 3	01/30/77	3.75
DAVIS-BESSE 1	08/28/77	3.18
DRESDEN 2	04/13/70	10.55
DRESDEN 3	07/22/71	9.28
DUANE ARNOLD	05/19/74	6.46
FARLEY 1	08/18/77	3.21
FITZPATRICK	02/01/75	5.75
FORT CALHOUN 1	08/25/73	7.19
FORT ST VRAIN	12/11/76	3.89
GINNA	12/02/69	10.92
HADDAM NECK	08/07/67	13.24
HATCH 1	11/11/74	5.97
HATCH 2	09/22/78	2.11
INDIAN POINT 2	06/26/73	7.35
INDIAN POINT 3	04/27/76	4.51
KEWAUNEE	04/08/74	6.57
LA CROSSE	04/26/68	12.52
MAINE YANKEE	11/08/72	7.98
MILLSTONE 1	11/29/70	9.92
MILLSTONE 2	11/09/75	4.98
MONTICELLO	03/05/71	9.66
NINE MILE POINT 1	11/09/69	10.98
NORTH ANNA 1	04/17/78	2.54
NORTH ANNA 2	08/25/80	.19
OCONEE 1	05/06/73	7.49
OCONEE 2	12/05/73	6.91
OCONEE 3	09/01/74	6.17
OYSTER CREEK 1	09/23/69	11.11
PALISADES	12/31/71	8.84
PEACH BOTTOM 2	02/18/74	6.70
PEACH BOTTOM 3	09/01/74	6.17
PILGRIM 1	07/19/72	8.29
POINT BEACH 1	11/06/70	9.99
POINT BEACH 2	08/02/72	8.25
PRAIRIE ISLAND 1	12/04/73	6.91
PRAIRIE ISLAND 2	12/21/74	5.86
QUAD CITIES 1	04/12/72	8.56
QUAD CITIES 2	05/23/72	8.44
RANCHO SECO 1	10/13/74	6.05
ROBINSON 2	09/26/70	10.10
SALEM 1	12/25/76	3.85

LICENSED OPERATING ELECTRICAL PRODUCING UNITS - CONT.

UNIT NAME	FIRST ELEC. GENERATION	YEARS EXPERIENCE
SAN ONOFRE 1	07/16/67	13.30
SEQUOYAH 1	07/22/80	.28
ST LUCIE 1	05/07/76	4.49
SURRY 1	07/04/72	8.33
SURRY 2	03/10/73	7.65
THREE MILE ISLAND 1	06/19/74	6.37
TROJAN	12/23/75	4.86
TURKEY POINT 3	11/02/72	8.00
TURKEY POINT 4	06/21/73	7.36
VERMONT YANKEE 1	09/20/72	8.11
YANKEE-ROWE 1	11/10/60	19.98
ZION 1	06/28/73	7.35
ZION 2	12/26/73	6.85
TOTAL		477.03 YEARS

PERMANENTLY OR INDEFINITELY SHUTDOWN UNITS

UNIT NAME	FIRST ELEC. GENERATION	SHUTDOWN OR ELECTRICAL CESSATION DATE	YEARS EXPERIENCE
BONUS	08/14/64	06/01/68	3.80
CVTR	12/18/63	01/01/67	3.04
DRESDEN 1	04/15/60	10/31/78	18.54
ELK RIVER	08/24/63	02/01/68	4.44
FERMI 1	08/05/66	11/29/72	6.32
HALLAM	05/29/63	09/01/64	1.26
HUMBOLDT BAY	04/18/63	07/02/76	13.21
INDIAN POINT 1	09/16/62	10/31/74	12.12
PATHFINDER	07/25/66	10/01/67	1.19
PEACH BOTTOM 1	01/27/67	11/01/74	7.76
PIQUA	11/04/63	01/01/66	2.16
THREE MILE ISLAND 2	04/21/78	03/28/79	.93
TOTAL			74.77 YEARS

NON–POWER REACTORS IN THE U.S.

1. RESEARCH REACTORS

State	Location	Licensee	Reactor Type	Docket	License Number	Date Issued	Authorized Power Level (KW)
Alabama	1. Tuskegee	Tuskegee Institute	AGN 201 No. 102	50-406	R-122	08-30-74	.0001
Arizona	2. Tucson	U. of Arizona	TRIGA Mk. I	50-113	R-52	12-05-58	100
California	3. Los Angeles	U. of California	Argonaut	50-142	R-71	10-03-60	100
	4. Berkeley	U. of California	TRIGA Mark III	50-224	R-101	08-10-66	1000
	5. Irvine	U. of California	TRIGA Mark I	50-326	R-116	11-24-69	250
	6. Santa Barbara	U. of California	L-77	50-433	R-124	12-03-74	.01
	7. Canoga Park	Rockwell International Corporation	L-77	50-94	R-40	05-17-58	.01
	8. Hawthorne	Northrop Corporate Laboratories	TRIGA Mark F	50-137	R-90	03-04-63	1000
	9. San Luis Obispo	California State Polytechnic College	AGN 201 No. 100	50-394	R-121	05-16-73	.0001
	10. Alameda County	General Electric Co.	NTR	50-73	R-33	10-31-57	100
	11. San Diego	General Atomic Co.	TRIGA Mark I	50-89	R-38	05-03-58	250
	12. San Diego	General Atomic Co.	TRIGA Mark F	50-163	R-67	07-01-60	1500
	13. San Ramon	Aerotest Operations, Inc.	TRIGA (Industrial)	50-228	R-98	07-02-65	250
Colorado	14. Jefferson County	U.S. Geological Survey. Dept. of Interior	TRIGA Mark I	50-274	R-113	02-24-69	1000
Delaware	15. Newark	U. of Delaware	AGN-201 No. 113	50-98	R-43	07-03-58	.0001
District of Columbia	16. –	Catholic U.	AGN-201 No. 101	50-77	R-31	11-15-67	.0001
Florida	17. Gainesville	U. of Florida	Argonaut	50-83	R-56	05-21-59	100
Georgia	18. Atlanta	Georgia Institute of Technology	AGN-201 No. 104	50-276	R-111	04-19-68	.0001
	19. Atlanta	Georgia Institute of Technology	Heavy Water	50-160	R-97	12-29-64	5000
Idaho	20. Pocatello	Idaho State U.	AGN-201 No. 103	50-284	R-110	10-11-67	.0001
Illinois	21. Urbana	U. of Illinois	Advanced TRIGA	50-151	R-115	07-22-69	1500
	22. Urbana	U. of Illinois	LOPRA	50-356	R-117	12-27-71	10
	23. Zion	Westinghouse Electric Corporation	NTR	50-87	R-119	01-28-72	10
Indiana	24. West Lafayette	Purdue U.	Lockheed	50-182	R-87	08-16-62	10

RESEARCH REACTORS – CONT.

State	Location	Licensee	Reactor Type	Docket	License Number	Date Issued	Authorized Power Level (KW)
Iowa	25. Ames	Iowa State U.	UTR-10	50-116	R-59	10-16-59	10
Kansas	26. Lawrence	U. of Kansas	Lockheed	50-148	R-78	06-23-61	250
	27. Manhattan	Kansas State U.	TRIGA	50-188	R-88	10-16-62	250
Maryland	28. College Park	U. of Maryland	TRIGA	50-166	R-70	10-14-60	250
	29. Bethesda	Armed Forces Radiobiology Research Institute	TRIGA	50-170	R-84	06-26-62	1000
Massachusetts	30. Lowell	U. of Lowell	GE	50-223	R-125	12-24-74	1000
	31. Cambridge	Massachusetts Institute of Technology	Heavy Water Reflected	50-20	R-37	06-09-58	5000
	32. Worcester	Worcester Polytechnic Institute	GE	50-134	R-61	12-16-59	10
Michigan	33. Ann Arbor	U. of Michigan	Pool	50-2	R-28	09-13-57	2000
	34. Lansing	Michigan State U.	TRIGA Mark I	50-294	R-114	03-21-69	250
	35. Midland	Dow Chemical Co.	TRIGA Mark I	50-264	R-108	07-03-67	100
Missouri	36. Rolla	U. of Missouri	Pool	50-123	R-79	11-21-61	200
	37. Columbia	U. of Missouri	Tank	50-186	R-103	10-11-66	10,000
Nebraska	38. Omaha	V.A. Hospital	TRIGA	50-131	R-57	06-26-59	18
New Mexico	39. Albuquerque	U. of New Mexico	AGN-201M No. 112	50-252	R-102	09-17-66	.005
New York	40. Buffalo	State U. of New York	PULSTAR'	50-57	R-77	03-24-61	2000
	41. Sterling Forest	Union Carbide Corp.	Pool	50-54	R-81	09-07-61	5000
	42. New York City	Columbia U.	TRIGA Mark II	50-208	R-128	04-14-77	250
	43. Ithaca	Cornell U.	ZPR	50-97	R-89	12-11-62	.1
	44. Ithaca	Cornell U.	TRIGA Mk. II	50-157	R-80	01-11-62	100
	45. Riverdale	Manhattan College	Tank	50-199	R-94	03-24-64	.0001
North Carolina	46. Raleigh	North Carolina State U.	PULSTAR	50-297	R-120	08-25-72	1000
Ohio	47. Columbus	Ohio State U.	Pool	50-150	R-75	02-24-61	10
Oklahoma	48. Norman	U. of Oklahoma	AGN-211 No. 102	50-112	R-53	12-29-58	.015
Oregon	49. Corvallis	Oregon State U.	AGN-201 No. 114	50-106	R-51	11-13-58	.0001
	50. Corvallis	Oregon State U.	TRIGA Mark II	50-243	R-106	03-07-67	1000
	51. Portland	Reed Institute	TRIGA Mk. I	50-288	R-112	07-02-68	250
Pennsylvania	52. University Park	Pennsylvania State U.	TRIGA Mk. III	50-5	R-2	07-08-55	1000

NON–POWER REACTORS IN THE U.S.

RESEARCH REACTORS – CONT.

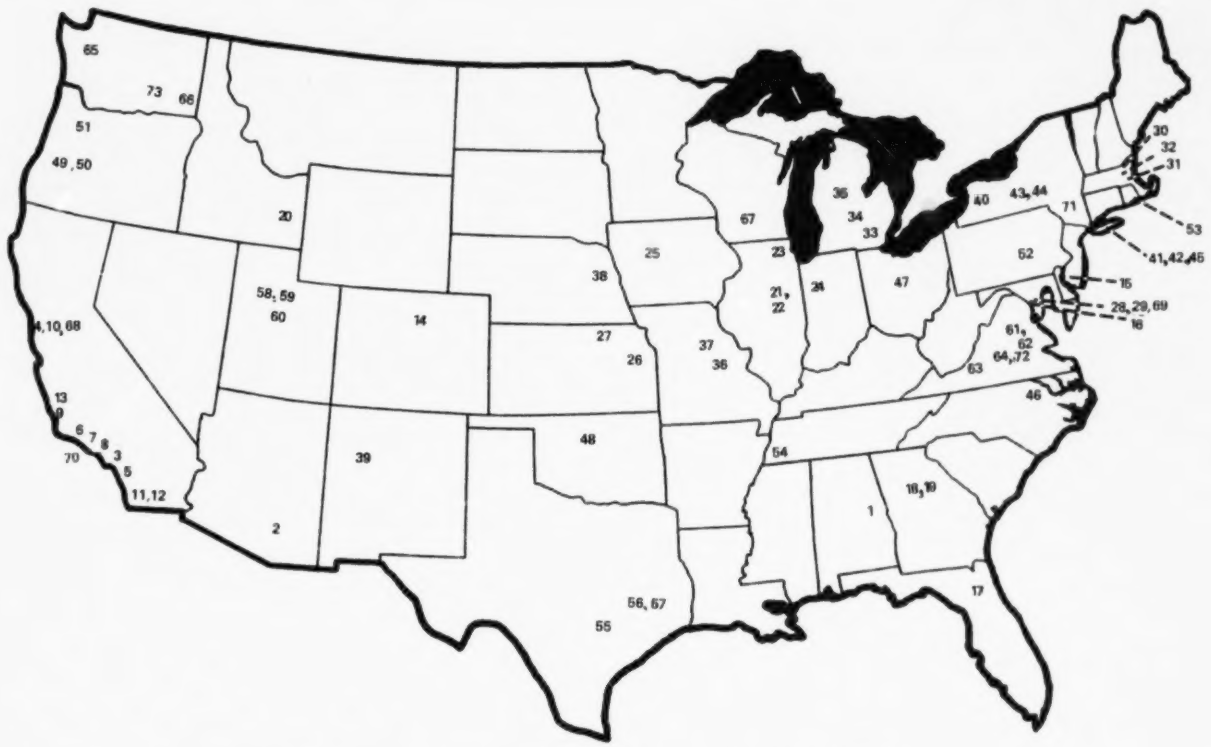
State	Location	Licensee	Reactor Type	Docket	License Number	Date Issued	Authorized Power Level (KW)
Rhode Island	53. Ft. Kearney, Narragansett	Rhode Island & Providence Plantations AEC	GE Pool	50-193	R-95	07-21-64	2000
Tennessee	54. Shelby County	Memphis State U.	AGN-201 No. 108	50-538	R-127	12-10-76	.0001
Texas	55. Austin	U. of Texas	TRIGA Mk. I	50-192	R-92	08-02-63	250
	56. College Station	Texas A&M U.	TRIGA	50-128	R-83	12-07-61	1000
	57. College Station	Texas A&M U.	AGN- 201M No. 106	50-59	R-23	08-26-57	.005
Utah	58. Salt Lake City	U. of Utah	AGN-201M No. 107	50-72	R-25	09-12-57	.005
	59. Salt Lake City	U. of Utah	TRIGA Mark I	50-407	R-126	09-30-75	100
	60. Provo	Brigham Young U.	L-77	50-262	R-109	09-07-67	.01
Virgina	61. Charlottesville	U. of Virgina	Pool	50-62	R-66	06-27-60	2000
	62. Charlottesville	U. of Virginia	Cavalier	50-396	R-123	09-24-74	.1
	63. Blacksburg	Virginia Polytechnic Institute	UTR-10	50-124	R-62	12-18-59	100
	64. Lynchburg	Babcock & Wilcox Co.	LPR	50-99	R-47	09-05-58	1000
Washington	65. Seattle	U. of Washington	Argonaut	50-139	R-73	03-31-61	100
	66. Pullman	Washington State U.	TRIGA	50-27	R-76	03-06-61	1000
Wisconsin	67. Madison	U. of Wisconsin	TRIGA	50-156	R-74	11-23-60	1000

2. EXPERIMENTAL AND TEST REACTORS

State	Location	Licensee	Reactor Type	Docket	License Number	Date Issued	Authorized Power Level (MWt)
California	68. Alameda Co.	General Electric Co.	GETR	50-70	TR-1	01-07-59	50
Maryland	69. Gaithersburg	National Bureau of Standards	TEST	50-184	TR-5	11-13-67 POL 06-30-70 OL	10

3. CRITICAL EXPERIMENT FACILITIES

State	Location	Licensee	Docket	License Number	Date Issued	Experiments
California	70. Ventura Co.	Rockwell International Corp. (Possession Only)	50-147	CX-17	10-03-60	AETR Experiments (Terminal Requested).
New York	71. Schenectady	Rensselaer Polytechnic Institute	50-225	CX-22	07-03-64	
Virginia	72. Lynchburg	Babcock & Wilcox Co.	50-13	CX-10	10-22-58	NMSR Experiments
Washington	73. Richland	Battelle Memorial Institute	50-360	CX-26	11-29-71	Plutonium Recycle Critical Facility



Source: NRC Facilities License Application Record (FLAR) 06-30-77.

ABBREVIATIONS

AGN	Aerojet General Nuclear	These reactors were constructed by Aeroject at San Ramon, California, and then transferred to the current location.
ZPR	Zero Power Reactor	
LPR	Low Power Reactor	

DISTRIBUTION LIST—NRC

INTERNAL DISTRIBUTION

Office of the Commissioners	5
Atomic Safety and Licensing Appeal Panel	3
Advisory Committee on Reactor Safeguards	6
Office of Inspector and Auditor	1
Office of Policy Evaluation	1
Office of the General Counsel	1
Office of Public Affairs	21
Office of Congressional Affairs	3
Office of the Executive Director for Operations	7
Office of Administration	2
Office of the Executive Legal Director	2
Office of the Controller	2
Office for Analysis and Evaluation of Operational Data	2
Office of International Programs	3
Office of State Programs	1
Office of Management and Program Analysis	14
Office of Standards Development	1
Office of Nuclear Material Safety and Safeguards	2
– Division of Fuel Cycle and Material Safety	3
– Division of Safeguards	2
Office of Nuclear Reactor Regulation Division of Licensing Division of Engineering Division of Systems Integration Division of Human Factors Safety Division of Safety Technology	138
Office of Nuclear Regulatory Research	4
Office of Inspection and Enforcement	7
– Region I	11
– Region II	10
– Region III	10
– Region IV	10
– Region V	7
	279

EXTERNAL DISTRIBUTION

SPECIAL REQUESTS

Air Force	1
Congress	10
Department of Energy	32
Government Accounting Office	1
Environmental Protection Agency	1
Electric Power Research Institute	1
Argonne National Laboratory	3
Federal Power Commission	1
Bureau of Mines	1
Department of Agriculture	1
Department of Commerce	1
Department of the Interior	1
	54

OTHER

GPO Depository	440
GPO Stores	75
National Technical Information Service	25
Subscriptions (NTIS)	250
Colleges and Libraries (including Public Document Rooms)	128
Utilities and Other Requests	227
	1145

15/4-12

Nuclear Regulatory Commission

OPERATING UNITS STATUS REPORT

DATA AS OF 11/30/80

NUREG 0020
VOL. 4 NO. 12
DECEMBER 1980

NEWARK PUBLIC LIBRARY
FEB 1 8 1981
U. S. GOVT. PUBS.
DEPOSITORY 376

LICENSED OPERATING REACTORS

Y3.N88:

Available from
GPO Sales Program
Division of Technical Information and Document Control
U.S. Nuclear Regulatory Commission
and
National Technical Information Service
Springfield, Virginia 22161
(703) 326-8543
A Year's Subscription, 12 Issues, for this Publication is
$120.00 North American Continent (240.00 Other).
Multiple Subscriptions Entered at the Same Time to the Same Address
Cost $90.00 North American Continent ($180.00 Other) for each Additional Subscription.

Single copies are $12.50 North American Continent ($25.00 Other).

This document is provided free of charge to those who participate in the data collection program, and to the Nuclear Regulatory Commission's local public document rooms.

Issued by the U.S. Nuclear Regulatory Commission
Washington, D.C. 20555

PUBLISHED: JANUARY 22, 1981

Nuclear Regulatory Commission

NUREG 0020
VOL. 4 NO. 12
DECEMBER 1980

OPERATING UNITS STATUS REPORT

DATA AS OF 11/30/80

LICENSED OPERATING REACTORS

AUTHORIZATION AND CLEARANCE

THE U.S. NUCLEAR REGULATORY COMMISSION'S OFFICE OF MANAGEMENT AND PROGRAM ANALYSIS PUBLISHES THIS MONTHLY STATUS REPORT "AS PART OF THE REPORTING REQUIREMENTS IN SECTION 50.36 OF 10 CFR PART 50 UNDER GAO CLEARANCE NUMBER B-180225, WITH AN EXPIRATION DATE OF SEPTEMBER 30, 1981," AS STATED IN THE OCTOBER 3, 1978 LETTER FROM JOHN M. LOVELADY, ASSISTANT DIRECTOR, GENERAL GOVERNMENT DIVISION, U. S. GENFRAL ACCOUNTING OFFICE, TO J.M. FELTON, DIRECTOR, DIVISION OF RULES AND RECORDS, U.S. NUCLEAR REGULATORY COMMISSION.

STATEMENT OF PURPOSE

The U. S. Nuclear Regulatory Commission's monthly OPERATING UNITS STATUS REPORT — LICENSED OPERATING REACTORS, provides data on the operation of nuclear units as timely and accurately as possible. This information is collected by the Office of Management and Program Analysis from the Headquarters Staff of NRC's Office of Inspection and Enforcement, from NRC's Regional Offices, and from utilities. Since all of the data concerning operation of the units is provided by the utility operators less than two weeks after the end of the month, necessary corrections to published information are shown on the ERRATA page.

This report is divided into three sections: the first contains monthly highlights and statistics for commercial operating units, and errata from previously reported data; the second is a compilation of detailed information on each unit, provided by NRC Regional Offices, IE Headquarters and the Utilities; and the third section is an appendix for miscellaneous information such as spent fuel storage capability, reactor-years of experience and non-power reactors in the United States.

The percentage computations, Items 19 through 24 in Section 2, the vendor capacity factors on page 1-4, and actual vs. potential energy production on Page 1-2 are computed from actual power production from the period of consideration. The percentages listed in power generation on Page 1-2 are computed as an arithmetical average. The factors for the life-span of each unit (the "Cumulative" column) are reported by the utility and are not entirely re-computed by NRC. Utility power production data is checked for consistency with previously submitted statistics.

In some cases, the numbers in the "Cumulative" column may have been totaled from a date that is different from the date of commercial operation. In these cases, the column is noted with an explanation of the nature of the change on each individual plant page in Section 2.

It is hoped this status report proves informative and helpful to all agencies and individuals interested in analyzing trends in the nuclear industry which might have safety implications, or in maintaining an awareness of the U. S. energy situation as a whole.

TABLE OF CONTENTS

GLOSSARY ii
INDEX TO LICENSED UNITS iii

SECTION 1 – CURRENT DATA SUMMARIES

MONTHLY HIGHLIGHTS OF COMMERCIAL NUCLEAR POWER UNITS 1-2

- Licensed Power Reactors 1-2
- Power Generation 1-2
- Actual vs. Potential Energy Production 1-2
- Outage Data 1-2
- Derated Units 1-2
- Shutdowns - Number and Reasons 1-3
- Unit Availability, Capacity, and Forced Outage Rate 1-3
- Refueling Schedule 1-3

AVERAGE CAPACITY FACTORS BY VENDOR 1-4

AVERAGE DAILY POWER LEVEL FOR ALL COMMERCIAL OPERATING UNITS 1-5

ERRATA 1-7

SECTION 2 – OPERATING POWER REACTORS

ARKANSAS 1 THROUGH ZION 2 PAGES A–2 TO Z–5

- Facility Data
- Inspection and Enforcement Status
- Reports Received from Licensee
- Operating Status
- Average Daily Power Level (MWe)
- Unit Shutdowns/Reductions

SECTION 3 – APPENDIX

STATUS OF SPENT FUEL STORAGE CAPABILITY 3-2
REACTOR-YEARS OF EXPERIENCE 3-3
NON-POWER REACTORS IN THE U.S. 3-4

GLOSSARY

AVERAGE DAILY POWER LEVEL (MWe)	The net electrical energy generated during the day (measured from 0001 to 2400 hours inclusive) in megawatts hours divided by 24 hours.
LICENSED THERMAL POWER (MWt)	The maximum thermal power of the reactor authorized by the NRC, expressed in megawatts.
DATE OF COMMERCIAL OPERATION	Date unit was declared by utility owner to be available for the regular production of electricity; usually related to satisfactory completion of qualification tests as specified in the purchase contract and to accounting policies and practices of utility.
DESIGN ELECTRICAL RATING (DER) (Net MWe)	The nominal net electrical output of the unit specified by the utility and used for the purpose of plant design.
FORCED OUTAGE	An outage required to be initiated no later than the weekend following discovery of an off-normal condition.
FORCED OUTAGE HOURS	The clock hours during the report period that a unit is unavailable due to forced outages.
GROSS ELECTRICAL ENERGY GENERATED (MWH)	Electrical output of the unit during the report period as measured at the output terminals of the turbine generator, in megawatts hours.
GROSS HOURS	The clock hours from the beginning of a specified situation until its end. For outage durations, the clock hours during which the unit is not in power production.
GROSS THERMAL ENERGY GENERATED (MWH)	The thermal energy produced by the unit during the report period as measured or computed by the licensee in megawatt hours.
HOURS GENERATOR ON-LINE	Also, "Unit Service Hours." The total clock hours in the report period during which the unit operated with breakers closed to the station bus. These hours added to the total outage hours experienced by the unit during the report period, shall equal the hours in the report period.
HOURS IN REPORTING PERIOD	For units in power ascension at the end of the period, the gross hours from the beginning of the period or the first electrical production, whichever comes last, to the end of the period. For units in commercial operation at the end of the period, the gross hours from the beginning of the period or of commercial operation, whichever comes last, to the end of the period or decommissioning, whichever comes first.
HOURS REACTOR CRITICAL	The total clock hours in the report period during which the reactor sustained a controlled chain reaction.
MAXIMUM DEPENDABLE CAPACITY (GROSS) (MDC Gross) (Gross MWe)	Dependable main-unit gross capacity, winter or summer, whichever is smaller. The dependable capacity varies because the unit efficiency varies during the year due to cooling water temperature variations. It is the gross electrical output as measured at the output terminals of the turbine generator during the most restrictive seasonal conditions (usually summer).
MAXIMUM DEPENDABLE CAPACITY (NET) (MDC Net) (Net MWe)	Maximum Dependable Capacity (Gross) less the normal station service loads.
NAMEPLATE RATING (Gross MWe)	The nameplate power designation of the generator in megavolt amperes (MVA) times the nameplate power factor of the generator. NOTE: The nameplate rating of the generator may not be indicative of the maximum or dependable capacity, since some other item of equipment of a lesser rating (e.g., turbine) may limit unit output.
NET ELECTRICAL ENERGY GENERATED	Gross electrical output of the unit measured at the output terminals of the turbine generator during the reporting period, minus the normal station service electrical energy utilization. If this quantity is less than zero, a negative number should be recorded.
OUTAGE	A situation in which no electrical production takes place.
OUTAGE DATE	As reported on Appendix D of Reg. Guide 1.16, the date of the start of the outage. If continued from a previous month, report the same outage date but change "Method of Shutting Down Reactor" to "4 (Other)" and add a note: "Continued from previous month."
OUTAGE DURATION	The total clock hours of the outage measured from the beginning of the report period or the outage, whichever comes last, to the end of the report period or the outage, whichever comes first.
OUTAGE NUMBER	A number unique to the outage assigned by the licensee. The same number is reported each month in which the outage is in progress. One format is "76-05" for the fifth outage to occur in 1976.
PERIOD HOURS	See "Hours in Reporting Period."
POWER REDUCTION	A reduction in the Average Daily Power Level of more than 20% from the previous day. All power reductions are defined as outage of zero hours durations for the purpose of computing unit service and availability factors, and forced outage rate.

REACTOR AVAILABLE HOURS	The total clock hours in the report period during which the reactor was critical or was capable of being made critical. (Reactor Reserve Shutdown Hours + Hours Reactor Critical.)
REACTOR AVAILABILITY FACTOR	$\frac{\text{Reactor Available Hours} \times 100}{\text{Period Hours}}$
REACTOR RESERVE SHUTDOWN	The cessation of criticality in the reactor for administrative or other similar reasons when operation could have been continued.
REACTOR RESERVE SHUTDOWN HOURS	The total clock hours in the report period during which the reactor is in reserve shutdown mode. NOTE: No credit is given for NRC imposed shutdowns.
REACTOR SERVICE FACTOR	$\frac{\text{Hours Reactor Critical} \times 100}{\text{Period Hours}}$
REPORT PERIOD	Usually, the preceding calendar month. Can also be the preceding calendar year, (Year-to-Date), or the life-span of a unit (cumulative).
RESTRICTED POWER LEVEL	Maximum net electrical generation to which the unit is restricted during the report period due to the state of equipment, external conditions, administrative reasons, or a direction by NRC.
SCHEDULED OUTAGE	Planned removal of a unit from service for refueling, inspection, training, or maintenance. Those outages which do not fit the definition of "Forced Outage" perforce are "Scheduled Outages."
STARTUP AND POWER ASCENSION TEST PHASE	Period following initial criticality during which the unit is tested at successively higher levels, culminating with operation at full power for a sustained period and completion of warranty runs. Following this phase, the utility generally considers the unit to be available for commercial operation.
UNIT	The set of equipment uniquely associated with the reactor, including turbine generators, and ancillary equipment, considered as a single electrical energy production facility.
UNIT AVAILABLE HOURS	The total clock hours in the report period during which the unit operated on-line or was capable of such operation. (Unit Reserve Shutdown Hours + Hours Generator On-Line.)
UNIT AVAILABILITY FACTOR*	$\frac{\text{Unit Available Hours} \times 100}{\text{Period Hours}}$
UNIT CAPACITY FACTORS	
– Using Licensed Thermal Power	$\frac{\text{Gross Thermal Energy Generated} \times 100}{\text{Period Hours} \times \text{Lic. Thermal Power}}$
– Using Nameplate Rating	$\frac{\text{Gross Electrical Energy Generated} \times 100}{\text{Period Hours} \times \text{Nameplate Rating}}$
– Using DER	$\frac{\text{Net Electrical Energy Generated} \times 100}{\text{Period Hours} \times \text{DER}}$
– Using MDC Gross	$\frac{\text{Gross Electrical Energy Generated} \times 100}{\text{Period Hours} \times \text{MDC Gross}}$
– Using MDC Net	$\frac{\text{Net Electrical Energy Generated} \times 100}{\text{Period Hours} \times \text{MDC Net}}$

NOTE: if MDC Gross and/or MDC Net have not been determined, the DER is substituted for this quantity for Unit Capacity Factor calculations.

UNIT FORCED OUTAGE RATE	$\frac{\text{Forced Outage Hours}}{\text{Unit Service Hours} + \text{Forced Outage Hours}}$
UNIT RESERVE SHUTDOWN	The removal of the unit from on-line operation for economic or other similar reasons when operation could have been continued.
UNIT RESERVE SHUTDOWN HOURS	The total clock hours in the report period during which the unit was in reserve shutdown mode.
UNIT SERVICE FACTOR	$\frac{\text{Unit Service Hours} \times 100}{\text{Period Hours}}$
UNIT SERVICE HOURS	See "Hours Generator On-Line."

INDEX TO LICENSED UNITS

ARKANSAS 1	A-3
ARKANSAS 2	A-5
BEAVER VALLEY 1	B-3
BIG ROCK POINT 1	B-5
BROWNS FERRY 1	B-7
BROWNS FERRY 2	B-9
BROWNS FERRY 3	B-11
BRUNSWICK 1	B-13
BRUNSWICK 2	B-15
CALVERT CLIFFS 1	C-3
CALVERT CLIFFS 2	C-5
COOK 1	C-7
COOK 2	C-9
COOPER STATION	C-11
CRYSTAL RIVER 3	C-13
DAVIS-BESSE 1	D-3
DRESDEN 1	D-5
DRESDEN 2	D-7
DRESDEN 3	D-9
DUANE ARNOLD	D-11
FARLEY 1	F-3
FITZPATRICK	F-5
FORT CALHOUN 1	F-7
FORT ST VRAIN	F-9
GINNA	G-3
HADDAM NECK	H-3
HATCH 1	H-5
HATCH 2	H-7
HUMBOLDT BAY	H-9
INDIAN POINT 2	I-3
INDIAN POINT 3	I-5
KEWAUNEE	K-3
LA CROSSE	L-3
MAINE YANKEE	M-3
MILLSTONE 1	M-5
MILLSTONE 2	M-7
MONTICELLO	M-9
NINE MILE POINT 1	N-3
NORTH ANNA 1	N-5
NORTH ANNA 2	N-7
OCONEE 1	O-3
OCONEE 2	O-5
OCONEE 3	O-7
OYSTER CREEK 1	O-9
PALISADES	P-3
PEACH BOTTOM 2	P-5
PEACH BOTTOM 3	P-7
PILGRIM 1	P-9
POINT BEACH 1	P-11
POINT BEACH 2	P-13
PRAIRIE ISLAND 1	P-15
PRAIRIE ISLAND 2	P-17
QUAD CITIES 1	Q-3
QUAD CITIES 2	Q-5
RANCHO SECO 1	R-3
ROBINSON 2	R-5
SALEM 1	S-3
SAN ONOFRE 1	S-5
SEQUOYAH 1	S-7
ST LUCIE 1	S-9
SURRY 1	S-11
SURRY 2	S-13
THREE MILE ISLAND 1	T-3
THREE MILE ISLAND 2	T-5
TROJAN	T-7
TURKEY POINT 3	T-9
TURKEY POINT 4	T-11
VERMONT YANKEE 1	V-3
YANKEE-ROWE 1	Y-3
ZION 1	Z-3
ZION 2	Z-5

SECTION 1 — CURRENT DATA SUMMARIES

MONTHLY HIGHLIGHTS OF COMMERCIAL NUCLEAR POWER UNITS

FOR MONTH OF

NOVEMBER 1980

LICENSED POWER REACTORS:

NUMBER		CAPACITY[1] MWe (Net)
66	IN COMMERCIAL OPERATION	48,724
2	IN POWER ASCENSION PHASE	2,038
68[2]	LICENSED TO OPERATE	50,762
2[3]	LICENSED FOR FUEL LOADING AND LOW POWER TESTING	

POWER GENERATION	REPORT MONTH	PREVIOUS MONTH	YEAR-TO-DATE
1. GROSS ELECTRICAL (MWhe)	21,801,728	25,177,099	238,428,895
2. NET ELECTRICAL (MWhe)	20,748,209	24,022,544	226,939,726
3. AVG. UNIT SERVICE FACTOR (%)	65.3	72.2	65.2
4. AVERAGE UNIT AVAILABILITY FACTOR (%)	65.3	72.5	65.3
5. AVERAGE UNIT CAPACITY FACTOR (MDC) (%)	57.7	65.4	57.6
6. AVG. UNIT CAPACITY FACTOR (DER) (%)	56.1	63.3	56.0
7. FORCED OUTAGE RATE (%)	11.6	8.2	11.3

NOTES

---

[1] BASED UPON MAXIMUM DEPENDABLE CAPACITY; DESIGN ELECTRIC RATING USED IF MDC NOT DETERMINED.

[2] EXCLUDES DRESDEN 1 (DER = 200), HUMBOLDT BAY (DER = 65), AND THREE MILE ISLAND 2 (DER = 906) WHICH HAVE OPERATING LICENSES BUT ARE SHUTDOWN INDEFINITELY.

[3] SALEM 2 (DER = 1115) WAS LICENSED ON APRIL 18, 1980
FARLEY 2 (DER=829) WAS LICENSED ON OCTOBER 23, 1980.

ACTUAL VS. POTENTIAL ENERGY PRODUCTION

			% OF POTENTIAL PRODUCTION
POTENTIAL ENERGY PRODUCTION IN THIS PERIOD BY UNITS IN COMMERCIAL OPERATION (Using Maximum Dependable Capacity Net)		35,081,280 MWHe	
ENERGY ACTUALLY PRODUCED DURING THIS REPORT PERIOD	=	20,748,209 MWHe NET	59.1
ENERGY NOT PRODUCED DURING THIS REPORT PERIOD DUE TO SCHEDULED OUTAGES (NET)	=	8,656,562 MWHe	24.7
ENERGY NOT PRODUCED DURING THIS REPORT PERIOD DUE TO FORCED OUTAGES (NET)	=	3,093,268 MWHe	8.8
ENERGY NOT PRODUCED DURING THIS REPORT PERIOD FOR OTHER REASONS (NET)	=	2,584,241 MWHe	7.4
TOTAL		35,081,280 MWHe =	100%
ENERGY NOT PRODUCED THIS REPORT PERIOD DUE TO NRC IMPOSED SHUTDOWNS AND RESTRICTIONS	=	1,424,311 MWHe	4.1
NUMBER OF UNITS WITH POWER RESTRICTIONS IMPOSED BY NRC	=	1	

OUTAGE DATA

	NUMBER	HOURS	PERCENT OF CLOCK TIME	MWHE LOST PRODUCTION
1. FORCED OUTAGES DURING MONTH	62	4,192.9	6.9	3,093,268
2. SCHEDULED OUTAGES DURING THE MONTH	31	12,279.9	20.1	8,656,562
3. TOTAL	93	16,472.8	26.9	11,748,830

MWHE LOST PRODUCTION = Down time x maximum dependable capacity net

DERATED UNITS

UNIT NAME	MAXIMUM DEPENDABLE CAPACITY (MWe - NET)	POWER LIMIT (MWe - NET)
COOPER	764	645
† FORT ST. VRAIN	330	231
INDIAN POINT 3	965	917
POINT BEACH 1	495	390
YANKEE – ROWE	175	150

† Restricted by NRC.

MONTHLY HIGHLIGHTS OF COMMERCIAL NUCLEAR POWER UNITS

FOR MONTH OF

NOVEMBER 1980

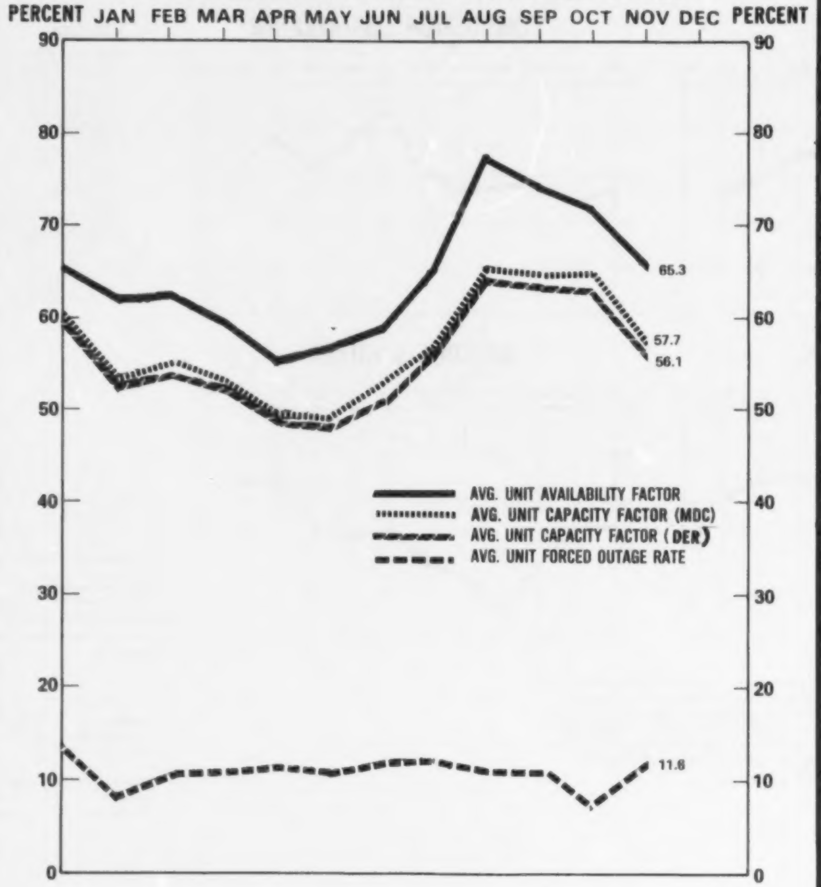
REASONS FOR SHUTDOWN

REASON	NUMBER	TOTAL HOURS LOST
A - Equipment Failure	41	3,703.1
B - Maintenance or Test	19	2,840.4
C - Refueling	15	7,940.1
D - NRC Restriction	3	1,754.1
E - Operator Training and License Examination	0	0
F - Administrative	0	0
G - Operational Error	11	189.7
H - Other	4	45.4
TOTAL	93	16,472.8

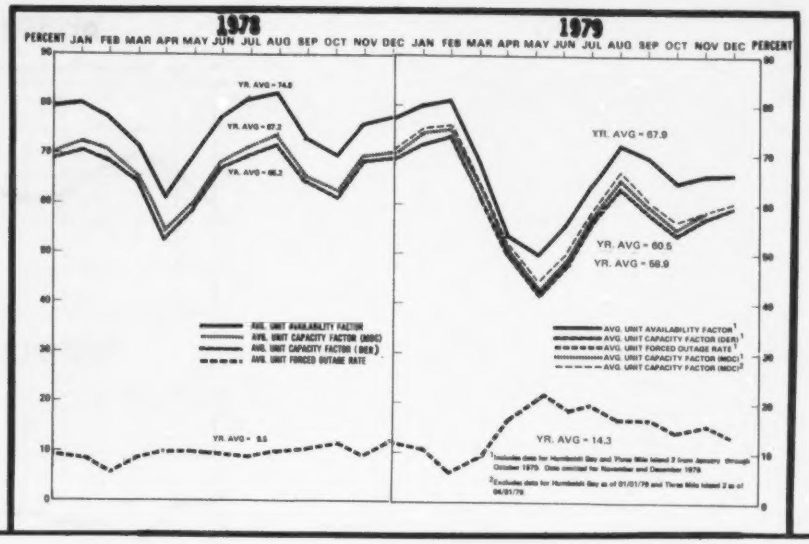
SHUTDOWNS GREATER THAN 72 HOURS EACH

UNIT NAME	REASON	UNIT NAME	REASON
BEAVER VALLEY 1	D	OCONEE 2	D
BIG ROCK POINT 1	C	OYSTER CREEK 1	B
BROWNS FERRY 2	C	PALISADES	B
BROWNS FERRY 3	C	PILGRIM 1	A
CALVERT CLIFFS 1	C	QUAD CITIES 1	C
COOK 2	A	QUAD CITIES 2	A
COOPER STATION	B	ROBINSON 2	A
DAVIS-BESSE 1	C	SALEM 1	C
DUANE ARNOLD	A	SAN ONOFRE 1	A
FARLEY 1	C	SURRY 1	A
GINNA	B	THREE MILE ISLAND 1	D
HATCH 2	C	TURKEY POINT 3	B
INDIAN POINT 2	C,A	TURKEY POINT 4	C
INDIAN POINT 3	B	VERMONT YANKEE 1	C,A
LA CROSSE	C	ZION 2	A
MILLSTONE 1	C		
MONTICELLO	B		

UNIT AVAILABILITY, CAPACITY AND FORCED OUTAGE



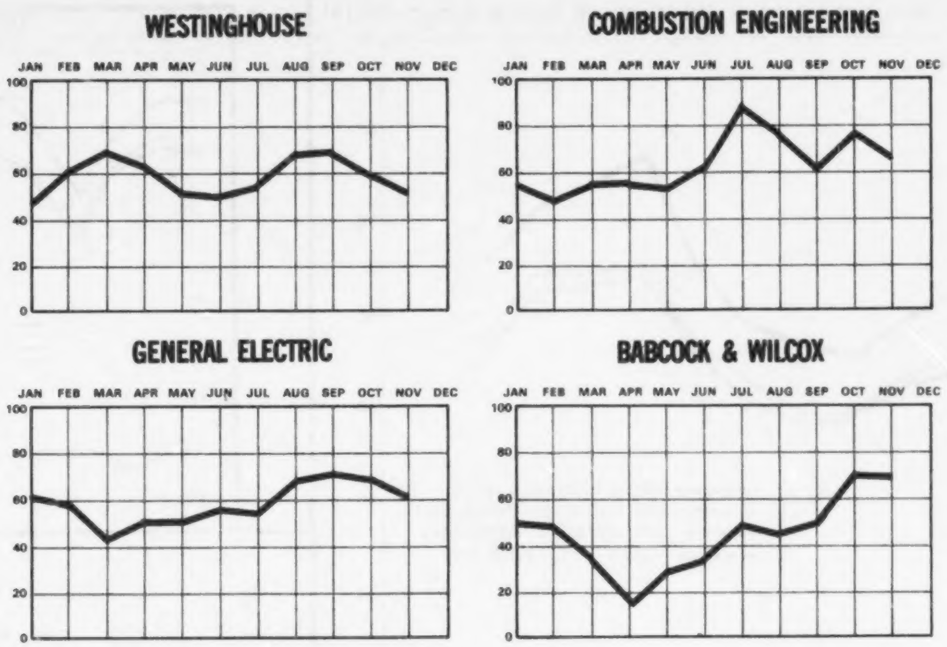
UNIT AVAILABILITY, CAPACITY AND FORCED OUTAGE



REFUELING SCHEDULE

Please refer to the table on page 3-2 for the scheduled refueling date for each nuclear powerplant in this publication.

VENDOR AVERAGE CAPACITY FACTORS



NOTE: THIS DISPLAY OF AVERAGE CAPACITY FACTORS PROVIDES A GENERAL PERFORMANCE COMPARISON OF PLANTS SUPPLIED BY THE FOUR NUCLEAR STEAM SUPPLY SYSTEM VENDORS. ONE MUST BE CAREFUL WHEN DRAWING CONCLUSIONS REGARDING THE REASONS FOR THE PERFORMANCE LEVELS INDICATED, SINCE PLANT PERFORMANCE MAY BE AFFECTED BY UNSPECIFIED FACTORS SUCH AS: (1) VARIOUS PLANT DESIGNS AND MODELS ARE INCLUDED FOR EACH VENDOR; (2) TURBINE/GENERATORS AND (3) DIFFERENT ARCHITECT/ENGINEERS ARE ALSO INVOLVED.

NOVEMBER 1980

OPERATIONS

GE BWR's	CFMDC
BROWNS FERRY 1	92.4
BROWNS FERRY 2	11.9
BROWNS FERRY 3	66.2
BRUNSWICK 1	86.8
BRUNSWICK 2	69.6
COOPER STATION	60.6
DRESDEN 2	43.7
DRESDEN 3	93.9
DUANE ARNOLD	56.9
FITZPATRICK	101.2
HATCH 1	75.0
HATCH 2	.0
MILLSTONE 1	.0
MONTICELLO	69.4
NINE MILE POINT 1	91.4
OYSTER CREEK 1	66.4
PEACH BOTTOM 2	82.4
PEACH BOTTOM 3	99.7
PILGRIM 1	68.4
QUAD CITIES 1	.0
QUAD CITIES 2	71.5
VERMONT YANKEE 1	.0

22 UNITS

W PWR's	
BEAVER VALLEY 1	3.7
COOK 1	98.7
COOK 2	.0
FARLEY 1	19.2
GINNA	0.0
HADDAM NECK	93.6
INDIAN POINT 2	.0
INDIAN POINT 3	.0
KEWAUNEE	99.9
NORTH ANNA 1	89.8
POINT BEACH 1	63.3
POINT BEACH 2	98.9
PRAIRIE ISLAND 1	100.2
PRAIRIE ISLAND 2	99.4
ROBINSON 2	88.0
SALEM 1	.0
SAN ONOFRE 1	.0
SURRY 1	.0
SURRY 2	95.9
TROJAN	98.8
TURKEY POINT 3	76.6
TURKEY POINT 4	21.6
YANKEE-ROWE 1	81.4
ZION 1	86.3
ZION 2	73.1

25 UNITS

CE PWR's	CFMDC
ARKANSAS 2	56.1
CALVERT CLIFFS 1	.0
CALVERT CLIFFS 2	89.2
FORT CALHOUN 1	72.4
MAINE YANKEE	96.6
MILLSTONE 2	94.1
PALISADES	0.0
ST LUCIE 1	100.2

8 UNITS

B&W PWR's	CFMDC
ARKANSAS 1	86.2
CRYSTAL RIVER 3	97.4
DAVIS-BESSE 1	48.2
OCONEE 1	101.5
OCONEE 2	21.1
OCONEE 3	91.7
RANCHO SECO 1	97.6
THREE MILE ISLAND 1	.0

8 UNITS

	GE BWR's	W PWR's	CE PWR's	B&W PWR's	All PWR's
NET ELECTRICAL PRODUCTION	7,559,733	6,909,050	2,838,795	3,316,675	13,064,520
MDC NET	17,164	18,312	6,069	6,737	31,118
CFMDC	61.2	52.4	65.0	68.4	58.3

Capacity factor in this page, denoted as CFMDC, is a function of the net maximum dependable capacity. See the corresponding definition in the Glossary. The vendor averages are computed by the formula :

$$\frac{\text{Net Electrical Energy Produced by Vendor}}{\text{Potential Electrical Production By Vendor In This Month.}} \times 100\%$$

Units Excluded	Reason for Exclusion
Dresden 1 Humboldt Bay Three Mile Island 2	Unit Shutdown Indefinitely
Big Rock Point	Not Representative of Typical BWRs
LaCrosse	Not Representative of Typical BWRs and Atypical Vendor
Fort St. Vrain	Atypical Vendor and Reactor Design

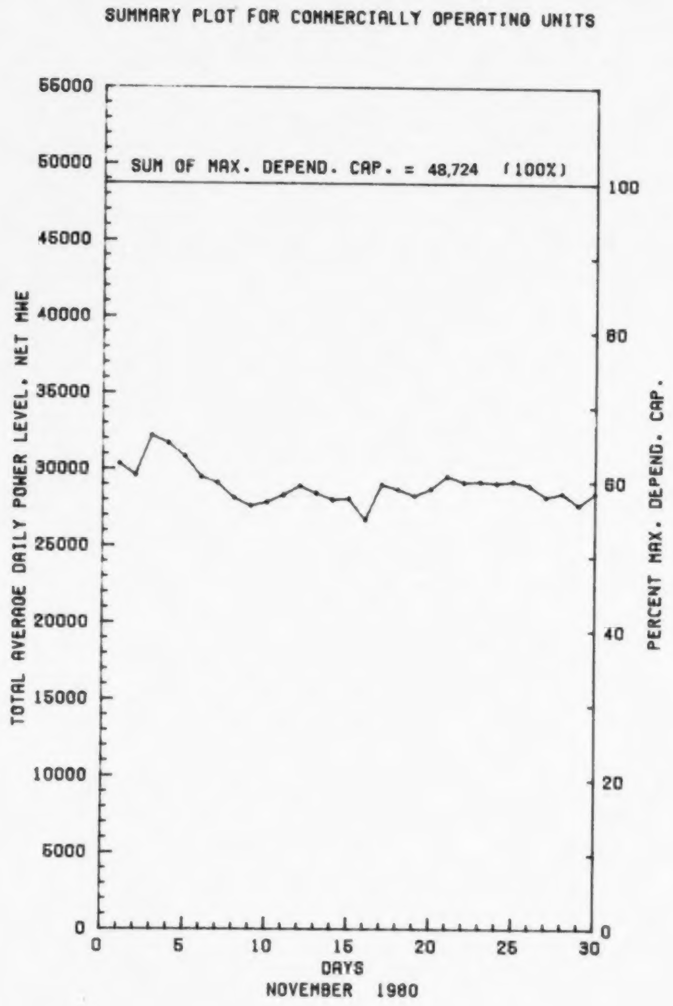
AVERAGE DAILY POWER LEVEL FOR ALL COMMERCIALLY OPERATING UNITS

This chart depicts the average daily power level for the units in commercial operation during the month.

The straight line on the graph labelled "SUM OF MAX. DEPEND. CAP." is plotted at the value shown by summing the separate maximum dependable capacities of the commercially operating units (in Net MWe). The plot shown below the line is calculated by summing the separate average daily power levels of the commercially operating units for each day of the month.

The scale on the left vertical axis runs in 1,000 MWe increments from 0 to 55,000 MWe (Net). The right vertical axis shows the percentage in 10% increments, up to 100% of the "SUM OF MAX. DEPEND. CAP."

It should be recognized that the 100% line would be obtainable only if all of the commercially operating units operated at 100% capacity, 24 hours per day, for the entire month. In other words, since any power generator must occasionally shut down to refuel and/or perform needed maintenance, and also since 100% capacity production is not always required by power demands, the 100% line is a theoretical goal and not a practical one.



ERRATA

Corrections and Amendments to Previously-Reported Data

(NOTE: These changes are reflected in the data contained in the current report)

CORRECTIONS TO NUREG-0020 VOL. 4 NO. 11 NOVEMBER 1980

Page	Unit Name	Item	Correction
1-4	BABCOCK & WILCOX	MONTHLY PLOT	THE PLOT OF VENDOR AVERAGE CAPACITY FACTORS FOR BABCOCK & WILCOX WAS INADVERTENTLY DRAWN AT ABOUT THE 50% LEVEL FOR THE MONTH OF OCTOBER. A CORRECT LINE WOULD HAVE BEEN TO THE 70.8% LEVEL AS SHOWN IN THE TABLE.

MULTIPLE - ISSUE CORRECTIONS

FORT ST. VRAIN

16. GROSS THERMAL ENERGY GENERATED - THE VALUE FOR APRIL 1980 SHOULD BE 235,462 RATHER THAN THE PUBLISHED VALUE OF 245,462.

ROBINSON-2

9. POWER LEVEL TO WHICH RESTRICTED - DURING CORE CYCLE 7, THE PERIOD FROM SEPTEMBER 1979 THROUGH AUGUST 1980, A POWER RESTRICTION OF 2200 MW THERMAL WAS REPORTED. THIS RESTRICTION, WHICH WAS DUE TO EXCESSIVE MOISTURE CARRY-OVER TO THE H. P. TURBINE, SHOULD HAVE BEEN REPORTED AS 2140 MW THERMAL.

SECTION 2 – OPERATING POWER REACTORS

NRC POWER RESTRICTIONS: NONE

ARKANSAS 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Arkansas/Pope 6 miles NW of Russellville
2.	Electric Reliability Council	Southwest Power Pool
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2568
5.	Design Electrical Rating (MWE Net)	850
6.	Date of Initial Criticality	August 6, 1974
7.	Date of Commercial Operation	December 19, 1974
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Dardenelle Reservoir

Utility & Contractor Information

10.	Licensee	Arkansas Power and Light Company
11.	Corporate Address	Ninth & Louisiana Streets Little Rock, Arkansas 72203
12.	Corporate Contact	W. Cavanaugh III Vice Pres., Generation & Construction
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	B&W
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	IV
17.	IE Branch Chief	G. L. Madsen
18.	IE Section Chief	D. M. Hunnicutt
19.	IE Resident Inspector	W. D. Johnson (501) 968-3290
20.	Licensing Project Manager	G. S. Vissing
21.	Docket Number	50-313
22.	License Number/Date of Issue	DPR-51, 5/21/74
23.	Public Document Room	Arkansas Polytechnic College Russellville, Arkansas 72801

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

FACILITY ITEMS (PLANS AND PROCEDURES):

MANAGERIAL ITEMS

PLANT STATUS:

POWER OPERATIONS.

LAST IE SITE INSPECTION DATE: SEPTEMBER 21 - OCTOBER 22, 1980

INSPECTION REPORT NO: 50-313/80-21

SUMMARY

INSPECTION SUMMARY

+ INSPECTION CONDUCTED DURING PERIOD OF JUNE 17-20, 1980 AND JULY 14-18, 1980 (80-12): ROUTINE, UNANNOUNCED INSPECTION OF LICENSEE ACTION ON PREVIOUS INSPECTION FINDINGS; LICENSEE INTERNAL AUDITS; QUALITY ASSURANCE PROGRAM FOR PROCUREMENT, RECEIPT, HANDLING AND STORAGE OF MATERIAL; MAINTENANCE RECORDS; LICENSED OPERATOR REQUALIFICATION PROGRAM; FIRE BRIGADE TRAINING; GENERAL TRAINING; AND TRAINING RECORDS. OF THE EIGHT (8) AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE FOUND IN FOUR (4) AREAS; FOUR (4) APPARENT ITEMS OF NONCOMPLIANCE WERE FOUND IN FOUR (4) AREAS (INFRACTION - FAILURE TO TAKE PROMPT CORRECTIVE ACTION, INFRACTION - REQUALIFICATION TRAINING PROGRAM REQUIREMENTS NOT MET, INFRACTION - FIRE BRIGADE TRAINING NOT COMPLETE, DEFICIENCY - FAILURE TO MAINTAIN RECORDS.

INSPECTION CONDUCTED DURING PERIOD SEPTEMBER 2-5, 1980 (80-16): ROUTINE, UNANNOUNCED INSPECTION INCLUDING AUDIT PROGRAMS, RECORDS, AND DOCUMENT CONTROL. OF THE THREE AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE WERE IDENTIFIED IN TWO AREAS; ONE APPARENT ITEM OF NONCOMPLIANCE (INFRACTION - FAILURE TO FOLLOW PROCEDURES WITH REGARD TO AUDIT FINDINGS WAS IDENTIFIED IN ONE AREA.

INSPECTION CONDUCTED DURING PERIOD OF AUGUST 22 - SEPTEMBER 21, 1980 (80-17): ROUTINE, ANNOUNCED INSPECTION INCLUDING SURVEILLANCE OBSERVATION MAINTENANCE, OPERATIONAL SAFETY VERIFICATION, INSPECTION DURING LONG-TERM SHUTDOWN, REVIEW OF PLANT OPERATIONS, FOLLOW-UP ON IE BULLETINS AND FOLLOW UP ON PREVIOUSLY IDENTIFIED ITEMS. NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION CONDUCTED DURING PERIOD SEPTEMBER 22-26, 1980 (80-18): ROUTINE, UNANNOUNCED INSPECTION INCLUDING SURVEILLANCE OF CORE POWER DISTRIBUTION LIMITS, CORE THERMAL POWER EVALUATION, FOLLOW-UP ON IE BULLETINS. WITHIN THE THREE (3) AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE AND ONE DEVIATION WERE IDENTIFIED (DEVIATION - SAMPLING PROGRAM).

INSPECTION CONDUCTED DURING PERIOD OF SEPTEMBER 22 - OCTOBER 21, 1980 (80-21): ROUTINE, ANNOUNCED INSPECTION INCLUDING SURVEILLANCE OBSERVATION MAINTENANCE, AND OPERATIONAL SAFETY VERIFICATION. IN THE THREE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

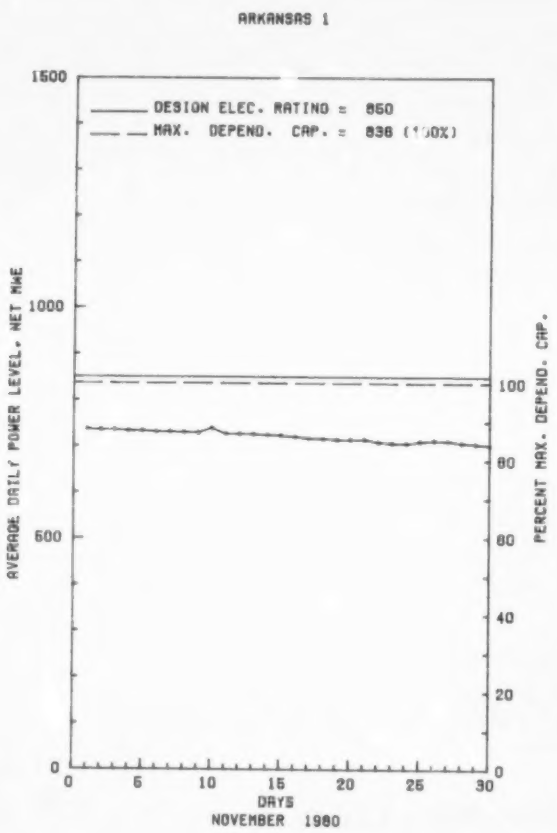
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-38 03L-0	10/13/80	11/11/80	DIESEL GENERATOR VOLTAGE REGULATOR FAILURE

REVIEWED BY OMPA LOEB: PE Robe **DATE:** 1/16/81

REVIEWED BY NRR DL: Guy S Vissing **DATE:** 1/12/81

ARKANSAS 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-313
2. Utility Contact: L. S. Bramlett (501) 968-2519
3. Licensed Thermal Power (MWt): 2568
4. Nameplate Rating (Gross MWe): 1003 x 0.9 = 903
5. Design Electrical Rating (Net MWe): 850
6. Maximum Dependable Capacity (Gross MWe): 883
7. Maximum Dependable Capacity (Net MWe): 836
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	8,040.0	52,171.0
12. Hours Reactor Was Critical	720.0	4,919.2	35,057.4
13. Reactor Reserve Shutdown Hours	.0	647.6	4,895.0
14. Hours Generator On-Line	720.0	4,850.7	34,321.2
15. Unit Reserve Shutdown Hours	.0	20.8	817.5
16. Gross Thermal Energy (MWH)	1,766,624	11,591,624	82,853,668
17. Gross Electrical Energy (MWH)	545,066	3,570,485	27,291,641
18. Net Electrical Energy (MWH)	518,626	3,395,448	26,029,476
19. Unit Service Facter	100.0	60.3	65.8
20. Unit Availability Facter	100.0	60.6	67.4
21. Unit Capacity Factor (MDC Net)	86.2	50.5	59.7
22. Unit Capacity Factor (DER Net)	84.7	49.7	58.7
23. Unit Forced Outage Rate	.0	27.8	17.6

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): JANUARY 2, 1981 TO MARCH 15, 1980, REFUELING
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
NONE									

SUMMARY

ARKANSAS 1 OPERATED NORMALLY WITH NO OUTAGES OR REDUCTIONS DURING NOVEMBER.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

ARKANSAS 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Arkansas/Pope 6 miles NW of Russellville
2.	Electric Reliability Council	Southwest Power Pool
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2815
5.	Design Electrical Rating (MWE Net)	912
6.	Date of Initial Criticality	12-5-78
7.	Date of Commercial Operation	3/26/80
8.	Condenser Cooling Method	Recirculating
9.	Condenser Cooling Water Source	Cooling Tower

Utility & Contractor Information

10.	Licensee	Arkansas Power & Light Company
11.	Corporate Address	Ninth & Louisiana Streets Little Rock, Arkansas 72203
12.	Corporate Contact	W. Cavanaugh III Vice Pres., Generation & Construction
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Combustion Engineering, Inc.
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	IV
17.	IE Branch Chief	G. L. Madsen
18.	IE Section Chief	D. M. Hunnicutt
19.	IE Resident Inspector	W. D. Johnson (501) 968-3290
20.	Licensing Project Manager	R. E. Martin
21.	Docket Number	50-368
22.	License Number/Date of Issue	NFP-6, 9/1/78
23.	Public Document Room	Arkansas Polytechnic College Russellville, Arkansas 72801

INSPECTION STATUS

ENFORCEMENT STATUS

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

FACILITY ITEMS (PLANS AND PROCEDURES):

MANAGERIAL ITEMS:

PLANT STATUS:

NO SIGNIFICANT ITEMS. POWER OPERATION

LAST IE INSPECTION DATE: SEPTEMBER 21 - OCTOBER 22, 1980

INSPECTION REPORT NO: 50-368/80-21

SUMMARY

INSPECTION SUMMARY

INSPECTION CONDUCTED DURING PERIOD OF JUNE 17-20, 1980 AND JULY 14-18, 1980 (80-12): ROUTINE, UNANNOUNCED INSPECTION OF LICENSEE ACTION ON PREVIOUS INSPECTION FINDINGS; LICENSEE INTERNAL AUDITS; QUALITY ASSURANCE PROGRAM FOR PROCUREMENT, RECEIPT, HANDLING AND STORAGE OF MATERIAL; MAINTENANCE RECORDS; LICENSED OPERATOR REQUALIFICATION PROGRAM; FIRE BRIGADE TRAINING; GENERAL TRAINING; AND TRAINING RECORDS; OF THE EIGHT (8) AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE FOUND IN FOUR (4) AREAS; FOUR (4) APPARENT ITEMS OF NONCOMPLIANCE WERE FOUND IN FOUR (4) AREAS (INFRACTION - FAILURE TO TAKE PROMPT CORRECTIVE ACTION. INFRACTION - REQUALIFICATION TRAINING PROGRAM REQUIREMENTS NOT MET. INFRACTION - FIRE BRIGADE TRAINING NOT COMPLETE. DEFICIENCY - FAILURE TO MAINTAIN RECORDS.

INSPECTION CONDUCTED DURING PERIOD SEPTEMBER 2-5, 1980 (80-16): ROUTINE, UNANNOUNCED INSPECTION INCLUDING AUDIT PROGRAMS, RECORDS, AND DOCUMENT CONTROL. OF THE THREE AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE WERE IDENTIFIED IN TWO AREAS; ONE APPARENT ITEM OF NONCOMPLIANCE (INFRACTION - FAILURE TO FOLLOW PROCEDURES WITH REGARD TO AUDIT FINDINGS - WAS IDENTIFIED IN ONE AREA.

INSPECTION CONDUCTED DURING PERIOD AUGUST 22 - SEPTEMBER 21, 1980 (80-17): ROUTINE, ANOUNNCED INSPECTION INCLUDING SURVEILLANCE OBSERVATION, MAINTENANCE. OPERATIONAL SAFETY VERIFICATION, INSPECTION DURING LONG-TERM SHUTDOWN, REVIEW OF PLANT OPERATIONS, FOLLOW-UP ON IE BULLETINS AND FOLLOW-UP ON PREVIOUSLY IDENTIFIED ITEMS.
NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION CONDUCTED DURING PERIOD OF SEPTEMBER 22-26, 1980 (80-18): ROUTINE, UNANNOUNCED INSPECTION INCLUDING SURVEILLANCE OF CORE POWER DISTRIBUTION LIMITS, CORE THERMAL POWER EVALUATION, FOLLOW-UP ON IE BULLETINS. WITHIN THE THREE (3) AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE AND ONE DEVIATION WERE IDENTIFIED (DEVIATION - SAMPLING PROGRAM).

INSPECTION CONDUCTED DURING PERIOD OF SEPTEMBER 22 - OCTOBER 21, 1980 (80-21): ROUTINE, ANNOUNCED INSPECTION INCLUDING OPERATIONAL SAFETY VERIFICATION. SURVEILLANCE OBSERVATION, MAINTENANCE, AND FOLLOW-UP ON LICENSEE EVENT REPORTS (LER). OF THE FOUR AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN ONE AREA, AND FOUR APPARENT ITEMS OF NONCOMPLIANCE WERE IDENTIFIED IN THREE AREAS. (VIOLATION, SEVERITY LEVEL IV, OPERABILITY OF HIGH PRESSURE SAFETY INJECTION PUMP, VIOLATION SEVERITY LEVEL V, FAILURE TO FOLLOW PROCEDURES, VIOLATION, SEVERITY LEVEL V, FAILURE TO SAMPLE THE REACTOR COOLANT SYSTEM AS REQUIRED, AND VIOLATION, SEVERITY LEVEL V, FAILURE TO REVISE PROCEDURES AND DRAWINGS FOLLOWING A DESIGN CHANGE.

REPORTS RECEIVED FROM LICENSEE

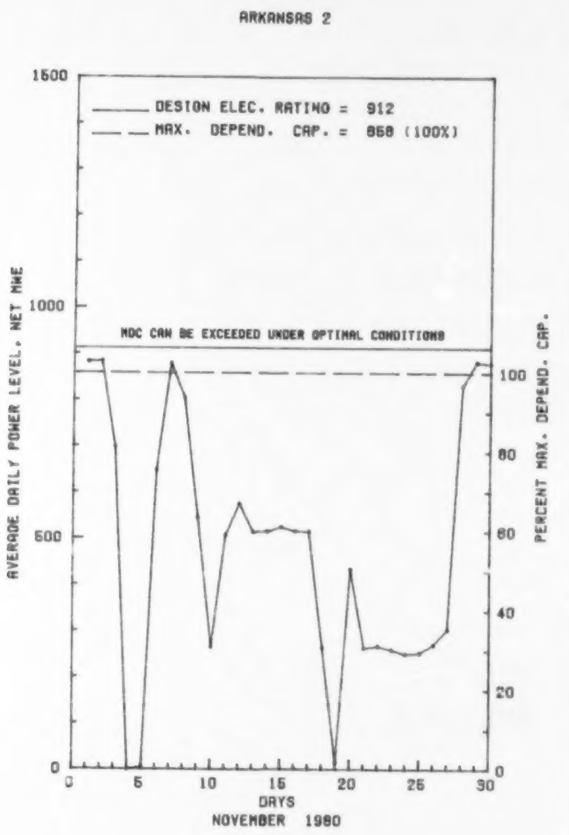
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-70 03L-0	10/5/80	11/4/80	CONTROL ROOM EMERGENCY CHILLER INOPERABLE
80-71 03L-0	10/6/80	11/4/80	WIND SPEED INDICATOR INOPERABLE
80-73 03L-0	10/2/80	10/31/80	FIRE MONITORING INSTRUMENTATION INOPERABLE
80-74 03L-0	10/4/80	11/3/80	STEAM GENERATOR LEVEL INDICATOR FAILURE
80-75 03L-0	10/7/80	11/5/80	SAFETY INJECTION TANK PRESSURE LOW
80-77 03L-0	10/7/80	11/5/80	SEISMIC ACCELEROGRAPH INOPERABLE
80-78 03L-0	10/14/80	11/11/80	HIGH PRESSURE SAFETY INJECTION PUMP FAILURE
80-79 03L-0	10/15/80	11/13/80	HIGH PRESSURE SAFETY INJECTION PUMP OPERABLE DEGRADED
80-80 03L-0	10/17/80	11/14/80	CONTROL ELEMENT ASSEMBLY CALCULATOR FAILURE

REVIEWED BY OMPA LOEB: MR Beebe **DATE:** 01/16/81

REVIEWED BY NRR DL: R Martin **DATE:** 01/16/81

ARKANSAS 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-368
2. Utility Contact: Rex Pendergraft (501) 968-2519
3. Licensed Thermal Power (MWt): 2815
4. Nameplate Rating (Gross MWe): 959
5. Design Electrical Rating (Net MWe): 912
6. Maximum Dependable Capacity (Gross MWe): 897
7. Maximum Dependable Capacity (Net MWe): 858
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	6,000.0	6,000.0
12. Hours Reactor Was Critical	661.0	4,603.1	4,603.1
13. Reactor Reserve Shutdown Hours	23.5	804.6	804.6
14. Hours Generator On-Line	641.0	4,518.8	4,518.8
15. Unit Reserve Shutdown Hours	.0	75.0	75.0
16. Gross Thermal Energy (MWH)	1,182,257	11,059,959	11,059,959
17. Gross Electrical Energy (MWH)	369,278	3,576,801	3,576,801
18. Net Electrical Energy (MWH)	346,565	3,405,397	3,405,397
19. Unit Service Factor	89.0	75.3	75.3
20. Unit Availability Factor	89.0	76.6	76.6
21. Unit Capacity Factor (MDC Net)	56.1	66.1	66.1
22. Unit Capacity Factor (DER Net)	52.8	62.2	62.2
23. Unit Forced Outage Rate	11.0	24.7	24.7

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
MARCH 15, 1981 TO MAY 15, 1981, REFUELING
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-24	11/03/80	F	49.5	A	1	80-086	CB	MECFUN	UNIT SHUTDOWN DUE TO EXCESSIVE RCS LEAKAGE. LEAK WAS ISOLATABLE. LEAK OCCURRED IN THE RCP DP SENSING LINE. FATIGUE WELD FAILURE WAS REPAIRED & UNIT RETURNED TO POWER.
80-25	11/09/80	F	.0	A	5				A FORCED POWER REDUCTION OCCURRED DUE TO A CONDENSER TUBE LEAK. LEAK WAS FOUND AND REPAIRED. DURATION OF POWER REDUCTION WAS APPROXIMATELY 219 HOURS.
80-26	11/18/80	F	29.5	A	1	80-084	SD	VALVEX	MSIV (2CV-1010) WAS DECLARED INOPERABLE DUE TO EXCESSIVE STROKE TIME. VALVE WAS ADJUSTED AND RETURNED TO OPERABLE STATUS.

SUMMARY

ARKANSAS 2 OPERATED WITH 2 OUTAGES AND 1 REDUCTION DUE TO EQUIPMENT FAILURE DURING NOVEMBER.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

NRC POWER RESTRICTIONS: NONE

BEAVER VALLEY 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Pennsylvania/Beaver 5 miles E of E. Liverpool, Ohio
2.	Electric Reliability Council	East Central Area Reliability Coordination Agreement
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2660
5.	Design Electrical Rating (MWE Net)	852
6.	Date of Initial Criticality	May 10, 1976
7.	Date of Commercial Operation	10/01/76 (50%) 04/30/77 (100%)
8.	Condenser Cooling Method	Cooling tower/Closed loop
9.	Condenser Cooling Water Source	Ohio River

Utility & Contractor Information

10.	Licensee	Duquesne Light Company
11.	Corporate Address	435 Sixth Avenue Pittsburgh, Pennsylvania 15219
12.	Corporate Contact	C. N. Dunn, Vice President
13.	Architect/Engineer	Stone & Webster
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Section Chief	E. C. MCCABE
19.	IE Resident Inspector	D. A. BECKMAN (412-643-9200/9201)
20.	Licensing Project Manager	D. A. Chaney
21.	Docket Number	50-334
22.	License Number/Date of Issue	DPR-66, 1/30/76
23.	Public Document Room	B. F. Jones Memorial Library 633 Franklin Avenue Alaquippa, PA 15001

INSPECTION STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

- OPERATION IS CONTINUING WITH TWO OF THREE PRESSURIZER POWER OPERATED RELIEF VALVE BLOCKED. THE LICENSEE HAS MADE COMMITMENTS TO PERFORM REPAIRS/MODIFICATIONS REQUIRED TO REFILL SAFETY AND RELIEF VALVE LOOP SEALS PENDING COMPLETION OF PIPING REANALYSES.
- ANALOG ROD POSITION INDICATOR INACCURACIES IDENTIFIED BY THE LICENSEE REQUIRED USE OF SPECIAL CALIBRATION TECHNIQUES AND ISSUANCE OF AN EMERGENCY TECHNICAL SPECIFICATION REVISION TO PERMIT USE OF DETECTOR VOLTAGES AS ALTERNATE METHOD OF ROD POSITION DETERMINATION.

FACILITY ITEMS (PLANS AND PROCEDURES):

- POST REFUELING CRITICALITY WAS ACHIEVED ON NOVEMBER 11. LOW POWER PHYSICS TESTING WAS COMPLETED ON NOVEMBER 17 AND THE UNIT WAS SYNCHRONIZED ON NOVEMBER 20.

PLANT STATUS:

- THE PLANT WAS OPERATING AT 60% FULL POWER AS OF DECEMBER 2. POWER ESCALATION WAS PENDING REPAIRS TO THE SECOND MAIN FEED PUMP.

- LAST IE SITE INSPECTION DATE: 11/3-7/80
- INSPECTION REPORT NO: 50-334/80-28

SUMMARY

INSPECTION SUMMARY

- 50-334/80-26 - AUG 18-22: SPECIAL, ANNOUNCED PERFORMANCE APPRAISAL INSPECTION BY AN APPRAISAL BRANCH INSPECTOR (34 HRS) OF THE LICENSEE'S PROGRAM AND DOCUMENTATION ASSOCIATED WITH THE CONTROL OF CENTRIFUGAL CHARGING/SAFETY INJECTION PUMPS MANUFACTURED BY PACIFIC PUMP DIVISION OF DRESSER INDUSTRIES. NO NONCOMPLIANCES WERE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

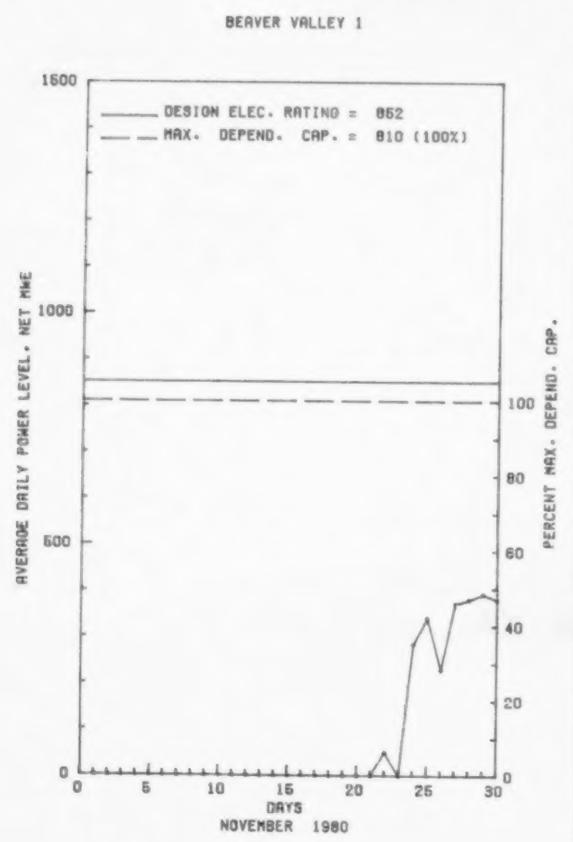
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-068/ 03L	09/17/80	10/17/80	TWO CHECK VALVES ON THE RIVER WATER SUPPLY TO THE CHARGING PUMP OIL COOLERS WERE FOUND INSTALLED BACKWARDS. THE VALVES WERE REPOSITIONED AND SIMILAR VALVES EXAMINED. NO FINDINGS
80-070/ 01T	09/22/80	09/30/80	INADVERTENT RCS DILUTION DUE TO DEFICIENT PROCEDURES. SHUT DOWN MARGIN ACCEPTABLY MAINTAINED. PROCEDURES HAVE BEEN CORRECTED.
80-072/ 03L	09/30/80	10/28/80	MAINTENANCE ON LEAKING 2B BORIC ACID PUMP IDENTIFIED HAIRLINE CRACK ON PUMP SHAFT DURING SEAL REPLACEMENT. SEAL AND SHAFT REPLACED.
80-074/ 03L	10/01/80	10/29/80	HI HEAD SI PUMP (CH-P-1C) DISCHARGE FLOW CHECK VALVE FAILED TO RESEAT AFTER SURVEILLANCE TEST DUE TO BINDING. VALVE REPLACED. ENGINEERING REVIEW UNDERWAY.
80-075/ 01T	10/08/80	10/21/80	POTENTIAL SEISMIC OVERSTRESS ON REACTOR COOLANT FILL HEADER FROM HIGH HEAD CHARGING PUMPS. MODIFICATION OF PIPE SUPPORT TO BE PERFORMED PRIOR TO STARTUP.
80-076/ 03L	11/02/80	11/20/80	SURVEILLANCE PERIOD EXCEEDED ON AIRLOCK SURVEILLANCE PRIOR TO ENTRY INTO MODE 4. TEST PERFORMED SATISFACTORILY. PERSONNEL COUNSELED. STARTUP PROCEDURE MODIFIED.
80-077/ 01P	11/14/80	11/17/80	IEB 79-14 RE-REVIEW IDENTIFIED OVERSTRESS CONDITION DURING NORMAL OPERATIONS ON 'A' MAIN STEAM HEADER BRANCH LINE. SUPPORT MODIFICATIONS WERE COMPLETED PRIOR TO STARTUP.
80-078/ 01P	11/14/80	11/17/80	REANALYZER OF PRESSURIZER PORV PIPING WITH WATER IN LOOP SEALS RESULTED IN NO OVERSTRESS CONDITION. SEALS WILL BE REESTABLISHED AT NEXT SCHEDULED SHUTDOWN OF SUFFICIENT DURATION.
80-082/ 03L	10/01/80	10/29/80	BOTH MAIN FILTER BANKS WERE FOUND ISOLATED AS A RESULT OF A DEFICIENT TEST PROCEDURE. FILTERS WERE UNISOLATED AND TEST PROCEDURE MODIFIED.

REVIEWED BY OMPA LOEB: PE Bobe **DATE:** 1/16/81

REVIEWED BY NRR DL: DAChaney **DATE:** 1/16/81

BEAVER VALLEY 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-334
2. Utility Contact: DAVID R. TIMKO (412) 643-5308
3. Licensed Thermal Power (MWt): 2660
4. Nameplate Rating (Gross MWe): 1026 x 0.9 = 923
5. Design Electrical Rating (Net MWe): 852
6. Maximum Dependable Capacity (Gross MWe): 845
7. Maximum Dependable Capacity (Net MWe): 810
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	8,040.0	40,200.0
12. Hours Reactor Was Critical	333.6	333.6	14,108.3
13. Reactor Reserve Shutdown Hours	.0	.0	4,482.7
14. Hours Generator On-Line	179.3	179.3	13,284.3
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	216,989	216,989	27,191,262
17. Gross Electrical Energy (MWH)	46,200	46,200	8,324,140
18. Net Electrical Energy (MWH)	21,816	21,816	7,592,559
19. Unit Service Factor	24.9	2.2	33.7
20. Unit Availability Factor	24.9	2.2	33.7
21. Unit Capacity Factor (MDC Net)	3.7	.3	26.0
22. Unit Capacity Factor (DER Net)	3.6	.3	24.7
23. Unit Forced Outage Rate	26.9	26.9	48.2

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
MAJOR MODIFICATION OUTAGE/MAY 15, 1981/30 DAYS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
1	01/01/80	S	474.7	D	4		ZZ	ZZZZZZ	UNIT SHUTDOWN FOR MAJOR MODIFICATIONS AS REQUIRED BY THE NRC, INCLUDING NRC BULLETINS IEB79-02 AND 79-14. (7794.7 HOURS TOTAL).
2	11/20/80	F	12.0	A	3		CH	INSTRU	1B & 1C STEAM GENERATOR BYPASS FEED CONTROL VALVES DID NOT REGULATE PROPERLY, CAUSING A HIGH STEAM GENERATOR LEVEL REACTOR TRIP.
3	11/21/80	F	30.3	A	3		CH	INSTRU	DUE TO A FAILED FEEDWATER FLOW SIGNAL ISOLATOR, THE 1B MAIN FEED REGULATING VALVE WENT WIDE OPEN DURING TRANSFER FROM BYPASS FLOW CONTROL CAUSING A HIGH STEAM GENERATOR LEVEL TRIP BEFORE THE OPERATOR COULD REGAIN MANUAL CONTROL.
4	11/23/80	F	17.0	A	3		HA	INSTRU	WHILE PERFORMING TURBINE THRUST BEARING OIL TRIP CHECK, THE TURBINE TRIPPED, WHICH RESULTED IN A REACTOR TRIP.
5	11/23/80	F	3.1	G	3		CH	INSTRU	OPERATOR UNFAMILIARITY WITH THE SLUGGISH CONTROL ACTION OF THE MAIN FEED REGULATING VALVES AND FEED BYPASS VALVES, WHEN TRANSFERRING FROM BYPASS FLOW CONTROL, CAUSED A LOW STEAM GENERATOR LEVEL TRIP.
6	11/26/80	F	3.7	G	3		HA	INSTRU	WHEN TROUBLESHOOTING AN ALARM PROBLEM WITH THE TURBINE EH SYSTEM, THE EH PANEL POWER SUPPLY WAS INADVERTANTLY SHORTED OUT, CAUSING A TURBINE TRIP AND A REACTOR TRIP.

SUMMARY

THE UNIT COMPLETED THE MODIFICATIONS REQUIRED BY I.E. BULLETINS 79-02 AND 79-14 AND STARTED POWER OPERATION FOR THE FIRST TIME THIS YEAR.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

BIG ROCK POINT 1

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Michigan/Charlevoix 4 miles NE of Charlevoix
2.	Electric Reliability Council	East Central Area Reliability Coordination Agreement
3.	Type of Reactor	BWR
4.	Power Level (MWT)	240
5.	Design Electrical Rating (MWE Net)	72
6.	Date of Initial Criticality	September 27, 1962
7.	Date of Commercial Operation	March 29, 1963
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10.	Licensee	Consumers Power Company
11	Corporate Address	212 West Michigan Avenue Jackson, Michigan 49201
12.	Corporate Contact	R. B. DeWitt
13.	Architect/Engineer	Bechtel Corporation
14.	Nuclear Steam Supply System	GE
15.	Constructor	Bechtel Corporation

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	D. C. BOYD
19.	IE Resident Inspector	G. Wright
20.	Licensing Project Manager	W. A. Paulson
21.	Docket Number	50-155
22.	License Number/Date of Issue	DPR-6 issued 8/30/62
23.	Public Document Room	Charlevoix Public Library 107 Clinton Street Charlevoix, Michigan

INSPECTION STATUS

ENFORCEMENT STATUS

```
   THE TRUSTWORTHINESS OF SOME CONTRACTORS GRANTED UNESCORTED ACCESS TO
   PROTECTED AND VITAL AREAS WAS BASED ON DOCUMENTS OTHER THAN THOSE
   REQUIRED BY THE SECURITY PLAN.
   (8012 INFRACTION)



    I N S P E C T I O N   S T A T U S

    OTHER ITEMS OF INTEREST

    PLANT STATUS

    UNIT SHUTDOWN ON OCTOBER 31, 1980, FOR A FIVE WEEK REFUELING OUTAGE.

*LAST IE SITE INSPECTION DATE:  SEPTEMBER 6 THROUGH OCTOBER 7, 1980

*          INSPECTION REPORT NO:  80-13

    SUMMARY

    INSPECTION SUMMARY

*   INSPECTION ON SEPTEMBER 6 THROUGH OCTOBER 7, (80-13):  ROUTINE, RESIDENT
    INSPECTOR INSPECTION OF OPERATIONAL SAFETY, LER REVIEW, SURVEILLANCE
    PROCEDURE REVIEW AND INDEPENDENT INSPECTION EFFORT.  THE INSPECTION
    INVOLVED 89 INSPECTOR-HOURS ONSITE BY ONE NRC INSPECTOR INCLUDING 4
    INSPECTOR-HOURS ONSITE DURING OFF-SHIFTS.  NO APPARENT ITEMS OF
    NONCOMPLIANCE WERE IDENTIFIED IN THE AREAS INSPECTED.
```

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

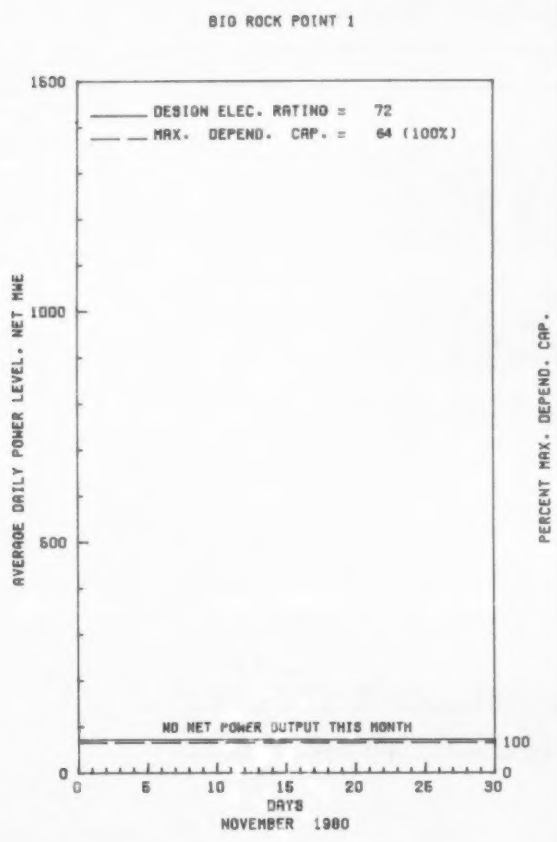
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-34/ 01T-0	11/01/80	11/19/80	MAIN STEAM ISOLATION VALVE MO 7050 FAILED TO CLOSE ON REMOTE MANUAL INITIATION.
80-35/ 03L-0	10/24/80	11/19/80	A SMALL AMOUNT OF WATER WAS NOTED TO BE LEAKING FROM A 4 INCH CIRCUMFERENTIAL CRACK IN THE ANION TANK DURING TRANSFER OF NEW RESINS FROM ANION TANK TO THE REACTOR CLEANUP DEMINERALIZER.

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 1-16-81

REVIEWED BY NRR DL: Walter A. Paulson **DATE:** 1-16-81

BIG ROCK POINT 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-155
2. Utility Contact: G. Himebauch (616) 547-6537 x180
3. Licensed Thermal Power (MWt): 240
4. Nameplate Rating (Gross MWe): 70.6 x 0.85 = 60
5. Design Electrical Rating (Net MWe): 72
6. Maximum Dependable Capacity (Gross MWe): 68
7. Maximum Dependable Capacity (Net MWe): 64
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

ITEM 21 CALCULATED WITH A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	8,040.0	154,963.0
12. Hours Reactor Was Critical	.0	6,960.1	107,093.1
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	.0	6,932.9	104,931.8
15. Unit Reserve Shutdown Hours	.0	0	.0
16. Gross Thermal Energy (MWH)	0	1,403,268	19,521,199
17. Gross Electrical Energy (MWH)	0	429,172	6,165,090
18. Net Electrical Energy (MWH)	0	405,450	5,833,787
19. Unit Service Factor	.0	86.2	67.7
20. Unit Availability Factor	.0	86.2	67.7
*21. Unit Capacity Factor (MDC Net)	.0	77.6	57.9
22. Unit Capacity Factor (DER Net)	.0	70.0	52.3
23. Unit Forced Outage Rate	.0	.6	22.3

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Data of Startup: 01/07/81

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
10	10/31/80	S	720.0	C	4				CONTINUATION OF REFUELING OUTAGE.

SUMMARY

THE UNIT CONTINUED IN THE REFUELING OUTAGE THAT WAS STARTED LAST MONTH.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

BROWNS FERRY 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Alabama/Limestone 10 miles NW of Decatur
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	3293
5.	Design Electrical Rating (MWE Net)	1065
6.	Date of Initial Criticality	August 17, 1973
7.	Date of Commercial Operation	August 1, 1974
8.	Condenser Cooling Method	Once Through With Forced Draft Cooling Towers
9.	Condenser Cooling Water Source	Tennessee River

Utility & Contractor Information

10.	Licensee	Tennessee Valley Authority
11.	Corporate Address	500A Chestnut Street Tower II Chattanooga, Tennessee 37401
12.	Corporate Contact	H. G. Parris, Manager of Power.
13.	Architect/Engineer	TVA
14.	Nuclear Steam Supply System	GE
15.	Constructor	TVA

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. Lewis
18.	IE Section Chief	H. C. Dance
19.	IE Resident Inspector	R. Sullivan (205) 729-6196
20.	Licensing Project Manager	R. Clark
21.	Docket Number	50-259
22.	License Number/Date of Issue	DPR-33, 6/26/73
23.	Public Document Room	Athens Public Library South and Forrest Athens, Alabama 35611

INSPECTION STATUS

ENFORCEMENT STATUS

AS REQUIRED BY TECHNICAL SPECIFICATION 3.11.E.2. AN INSPECTION AND AUDIT BY AN OUTSIDE QUALIFIED FIRE CONSULTANT WILL BE PERFORMED AT INTERVALS NO GREATER THAN 3 YEARS. CONTRARY TO THE ABOVE, ON SEPTEMBER 1, 1980, GREATER THAN THREE YEARS HAD ELASPED SINCE THE LAST FIRE INSPECTION AND AUDIT BY AN OUTSIDE AGENCY.
(8035 INFRACTION)

AS REQUIRED BY TECHNICAL SPECIFICATION 6.3.A.2. DETAILED WRITTEN PROCEDURES, INCLUDING APPLICABLE CHECK-OFF LISTS ARE REQUIRED FOR REFUELING OPERATION. GENERAL OPERATING INSTRUCTION (GOI) 100.3. REFUELING OPERATIONS, REQUIRES VERIFICATION OF PROPER FUEL ASSEMBLY ORIENTATION AFTER LOADING OF EACH FUEL ASSEMBLY PLUS A VERIFICATION OF EACH FUEL ASSEMBLY AFTER THE CORE HAS BEEN FULLY LOADED. IN ADDITION TECHNICAL INSTRUCTION-14 (TI-14), SPECIAL NUCLEAR MATERIALS CONTROL AND ACCOUNTABILITY SYSTEM, REQUIRES A FULL CORE VERIFICATION FOR PROPER FUEL ASSEMBLY ORIENTATION BY A MINIMUM OF TWO PARTIES. CONTRARY TO THE ABOVE, PERSONNEL FAILED TO VERIFY PROPER ORIENTATION OF TWO FUEL ASSEMBLIES ON UNIT 2 AS EVIDENCED ON SEPTEMBER 14, 1980, WHEN IT WAS DETERMINED THAT ON THE LAST REFUELING OUTAGE, IN MAY, 1979, TWO FUEL ASSEMBLIES WERE MISORIENTATED BY 90 DEGREES AND NOT DETECTED TILL AFTER THE NEXT REFUELING OUTAGE. IN ADDITION, ON SEPTEMBER 19, 1980 WITH UNIT 1 AT POWER, IT WAS DETERMINED BY A REVIEW OF THE CORE LOAD TAPES THAT ONE FUEL ASSEMBLY IS MISORIENTATED BY 90 DEGREES IN UNIT 1.
(8035 INFRACTION)

AS REQUIRED BY TECHNICAL SPECIFICATION 6.3.D.1, EACH HIGH RADIATION AREA IN WHICH INTENSITY OF RADIATION IS GREATER THAN 100 MREM/HR, ENTRANCE SHALL BE CONTROLLED BY ISSUANCE OF A SPECIAL WORK PERMIT. CONTRARY TO THE ABOVE, ON SEPTEMBER 21 AND 22, 1980, SIX TVA PERSONNEL ENTERED A HIGH RADIATION AREA MEASURING 300 MREM/HR, WITHOUT THE CONTROL OF A SPECIAL WORK PERMIT FOR ENTRANCE TO A HIGH RADIATION AREA.
(8035 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

100 PERCENT FULL POWER OPERATION.

+ LAST IE SITE INSPECTION DATE: OCTOBER 14-15, 1980

+ INSPECTION REPORT NO: 50-259/80-38

SUMMARY

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-072/ 03L-0	09/24/80	10/20/80	RESIDUAL HEAT REMOVAL TORUS RETURN VALVE INOPERABLE.
80-075/ 03L-0	10/06/80	11/04/80	REACTOR CORE ISOLATION COOLING STEAM SUPPLY VALVE INDICATION INOPERATIVE
80-076/ 03L-0	10/07/80	11/05/80	AIR DIFFERENTIAL TEMPERATURE RECORDER INOPERATIVE
80-080/ 03L-0	10/15/80	11/13/80	FIRE PROTECTION SYSTEM HAS INOPERATIVE VALVE

INSPECTION SUMMARY

INSPECTION SEPTEMBER 1-30 (80-35): THIS ROUTINE INSPECTION INVOLVED 30 RESIDENT INSPECTOR-HOURS ON SITE IN THE AREAS OF OPERATIONAL SAFETY, IE CIRCULAR FOLLOWUP, REPORTABLE OCCURRENCES, PLANT PHYSICAL PROTECTION, FIRE PROTECTION, REFUELING OUTAGE, REACTOR TRIP, IE BULLETIN FOLLOWUP, AND HEALTH PHYSICS. OF THE NINE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE FOUND IN SIX AREAS; THREE SEPARATE ITEMS OF NONCOMPLIANCE WERE FOUND IN THREE AREAS; (INFRACTION - FAILURE TO FOLLOW PROCEDURE FOR FUEL ASSEMBLY ORIENTATION VERIFICATION; INFRACTION - FAILURE TO HAVE AN OUTSIDE AGENCY PERFORM A FIRE INSPECTION WITHIN THREE YEARS; INFRACTION - FAILURE OF PERSONNEL TO HAVE A SPECIAL WORK PERMIT FOR ENTRY INTO A HIGH RADIATION AREA.

INSPECTION OCTOBER 6-7 (80-37): INCLUDED REVIEW OF LICENSEE IDENTIFIED ITEM OF NONCOMPLIANCE AND LICENSEE PROCEDURE FOR UNESCORTED ACCESS CLEARANCES. THE INSPECTOR ALSO REVIEWED THE LICENSEES INVESTIGATION OF A RECENT INTRUSION OF THE PROTECTED AREA AT BROWNS FERRY. THE INSPECTION INVOLVED THREE INSPECTOR-HOURS ON SITE BY ONE NRC INSPECTOR. THE INSPECTION WAS BEGUN DURING A REGULAR PERIOD. THE LICENSEE WAS FOUND TO BE IN COMPLIANCE WITH NRC REQUIREMENTS WITHIN THE AREAS EXAMINED DURING THE INSPECTION.

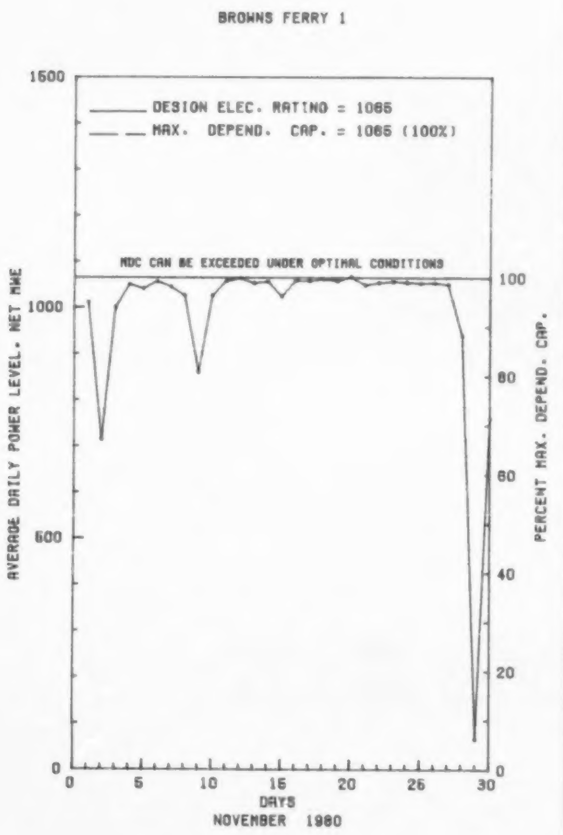
INSPECTION OCTOBER 14-15, (80-38): THIS SPECIAL, UNANNOUNCED INSPECTION INVOLVED THREE INSPECTOR-HOURS ON SITE IN THE AREA OF A LICENSEE IDENTIFIED ITEM CONCERNING SETTLING OF STRUCTURAL BACKFILL UNDER THE RHR SERVICE WATER TUNNELS. OF THE AREA INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATION WERE IDENTIFIED.

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 1-16-81

REVIEWED BY NRR DL: Dick Clark **DATE:** 01-19-81

BROWNS FERRY 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-259
2. Utility Contact: Don Green (205) 729-6846
3. Licensed Thermal Power (MWt): 3293
4. Nameplate Rating (Gross MWe): 1280 x 0.9 = 1152
5. Design Electrical Rating (Net MWe): 1065
6. Maximum Dependable Capacity (Gross MWe): 1098
7. Maximum Dependable Capacity (Net MWe): 1065
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	8,040.0	55,538.0
12. Hours Reactor Was Critical	706.0	5,800.4	34,077.7
13. Reactor Reserve Shutdown Hours	14.0	394.0	5,083.5
14. Hours Generator On-Line	700.7	5,657.7	33,270.6
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	2,160,958	16,912,943	91,049,436
17. Gross Electrical Energy (MWH)	727,060	5,502,960	30,021,420
18. Net Electrical Energy (MWH)	708,734	5,338,797	29,140,765
19. Unit Service Factor	97.3	70.4	59.9
20. Unit Availability Factor	97.3	70.4	59.9
21. Unit Capacity Factor (MDC Net)	92.4	62.4	49.3
22. Unit Capacity Factor (DER Net)	92.4	62.4	49.3
23. Unit Forced Outage Rate	2.7	8.0	29.8

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUELING: APRIL, 1981
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
154	11/01/80	S	.0	H	5				CONTROL ROD SEQUENCE CHANGE FROM "B" TO "A".
155	11/08/80	S	.0	H	5				CONTROL ROD PATTERN ADJUSTMENT.
156	11/15/80	S	.0	B	5				TURBINE C.V. TESTS AND SI'S.
157	11/28/80	F	19.3	B	3				TURBINE TRIP WHILE SHUTTING DOWN FOR MAINTENANCE (STEAM LEAKS).

SUMMARY

THERE WAS ONE SCRAM AND THREE POWER REDUCTIONS DURING THE MONTH. ON NOVEMBER 28, THE REACTOR SCRAMMED WHEN THE MAIN TURBINE TRIPPED DUE TO PROBLEMS IN THE TRANSFER OF THE SHUTDOWN BUS FROM UNIT 1 STATION SERVICE TO UNIT 2 STATION SERVICE IN PREPARATION FOR A MANUAL SCRAM FOR STEAM LEAK MAINTENANCE ON UNIT 1.

LEGEND

[1] Type:
F–Forced
S–Scheduled

[2] Reason:
A–Equipment Failure
B–Maintenance or Test
C–Refueling
D–Regulatory Restriction
E–Operator Training & License Examination
F–Administrative
G–Operational Error
H–Other

[3] Method:
1–Manual
2–Manual Scram
3–Automatic Scram
4–Continuations
5–Load Reduction
9–Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

BROWNS FERRY 2

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Alabama/Limestone 10 miles NW of Decatur
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	3293
5.	Design Electrical Rating (MWE Net)	1065
6.	Date of Initial Criticality	July 20, 1974
7.	Date of Commercial Operation	March 1, 1975
8.	Condenser Cooling Method	Once Through With Forced Draft Cooling Towers
9.	Condenser Cooling Water Source	Tennessee River

Utility & Contractor Information

10.	Licensee	Tennessee Valley Authority
11.	Corporate Address	500A Chestnut Street Tower II Chattanooga, Tennessee 37401
12.	Corporate Contact	H. G. Parris, Manager of Power
13.	Architect/Engineer	TVA
14.	Nuclear Steam Supply System	GE
15.	Constructor	TVA

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. Lewis
18.	IE Section Chief	H. C. Dance
19.	IE Resident Inspector	R. Sullivan (205) 729-6196
20.	Licensing Project Manager	R. Clark
21.	Docket Number	50-260
22.	License Number/Date of Issue	DPR-52, 6/28/74
23.	Public Document Room	Athens Public Library South and Forrest Athens, Alabama 35611

INSPECTION STATUS

ENFORCEMENT STATUS

AS REQUIRED BY TECHNICAL SPECIFICATION 3.11.E.2, AN INSPECTION AND AUDIT BY AN OUTSIDE QUALIFIED FIRE CONSULTANT WILL BE PERFORMED AT INTERVALS NO GREATER THAN 3 YEARS. CONTRARY TO THE ABOVE, ON SEPTEMBER 1, 1980, GREATER THAN THREE YEARS HAD ELASPED SINCE THE LAST FIRE INSPECTION AND AUDIT BY AN OUTSIDE AGENCY.
(8029 INFRACTION)

AS REQUIRED BY TECHNICAL SPECIFICATION 6.3.A.2, DETAILED WRITTEN PROCEDURES, INCLUDING APPLICABLE CHECK-OFF LISTS ARE REQUIRED FOR REFUELING OPERATION. GENERAL OPERATING INSTRUCTION (GOI) 100.3, REFUELING OPERATIONS, REQUIRES VERIFICATION OF PROPER FUEL ASSEMBLY ORIENTATION AFTER LOADING OF EACH FUEL ASSEMBLY PLUS A VERIFICATION OF EACH FUEL ASSEMBLY AFTER THE CORE HAS BEEN FULLY LOADED. IN ADDITION TECHNICAL INSTRUCTION-14 (TI-14), SPECIAL NUCLEAR MATERIALS CONTROL AND ACCOUNTABILITY SYSTEM, REQUIRES A FULL CORE VERIFICATION FOR PROPER FUEL ASSEMBLY ORIENTATION BY A MINIMUM OF TWO PARTIES. CONTRARY TO THE ABOVE, PERSONNEL FAILED TO VERIFY PROPER ORIENTATION OF TWO FUEL ASSEMBLIES ON UNIT 2 AS EVIDENCED ON SEPTEMBER 14, 1980, WHEN IT WAS DETERMINED THAT ON THE LAST REFUELING OUTAGE, IN MAY, 1979, TWO FUEL ASSEMBLIES WERE MISORIENTATED BY 90 DEGREES AND NOT DETECTED TILL AFTER THE NEXT REFUELING OUTAGE. IN ADDITION, ON SEPTEMBER 19, 1980 WITH UNIT 1 AT POWER, IT WAS DETERMINED BY A REVIEW OF THE CORE LOAD TAPES THAT ONE FUEL ASSEMBLY IS MISORIENTATED BY 90 DEGREES IN UNIT 1.
(8029 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

RESUMED POWER OPERATION AFTER COMPLETION OF REFUELING.

+ LAST IE SITE INSPECTION DATE: OCTOBER 14-15, 1980

+ INSPECTION REPORT NO: 50-260/80-32

SUMMARY

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE.			

INSPECTION SUMMARY

+ INSPECTION SEPTEMBER 1-30 (80-29): THIS ROUTINE INSPECTION INVOLVED 39 RESIDENT INSPECTOR-HOURS ON SITE IN THE AREAS OF OPERATIONAL SAFETY, IE CIRCULAR FOLLOWUP, REPORTABLE OCCURRENCES, PLANT PHYSICAL PROTECTION, FIRE PROTECTION, REFUELING OUTAGE, REACTOR TRIP, IE BULLETIN FOLLOWUP, AND HEALTH PHYSICS. OF THE NINE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE FOUND IN SIX AREAS; THREE SEPARATE ITEMS OF NONCOMPLIANCE WERE FOUND IN THREE AREAS; (INFRACTION - FAILURE TO FOLLOW PROCEDURE FOR FUEL ASSEMBLY ORIENTATION VERIFICATION; INFRACTION - FAILURE TO HAVE AN OUTSIDE AGENCY PERFORM A FIRE INSPECTION WITHIN THREE YEARS; INFRACTION - FAILURE OF PERSONNEL TO HAVE A SPECIAL WORK PERMIT FOR ENTRY INTO A HIGH RADIATION AREA.

INSPECTION OCTOBER 6-7 (80-31): INCLUDED REVIEW OF LICENSEE IDENTIFIED ITEM OF NONCOMPLIANCE AND LICENSEE PROCEDURE FOR UNESCORTED ACCESS CLEARANCES. THE INSPECTOR ALSO REVIEWED THE LICENSEES INVESTIGATION OF A RECENT INTRUSION OF THE PROTECTED AREA AT BROWNS FERRY. THE INSPECTION INVOLVED THREE INSPECTOR-HOURS ON SITE BY ONE NRC INSPECTOR. THE INSPECTION WAS BEGUN DURING A REGULAR PERIOD. THE LICENSEE WAS FOUND TO BE IN COMPLIANCE WITH NRC REQUIREMENTS WITHIN THE AREAS EXAMINED DURING THE INSPECTION.

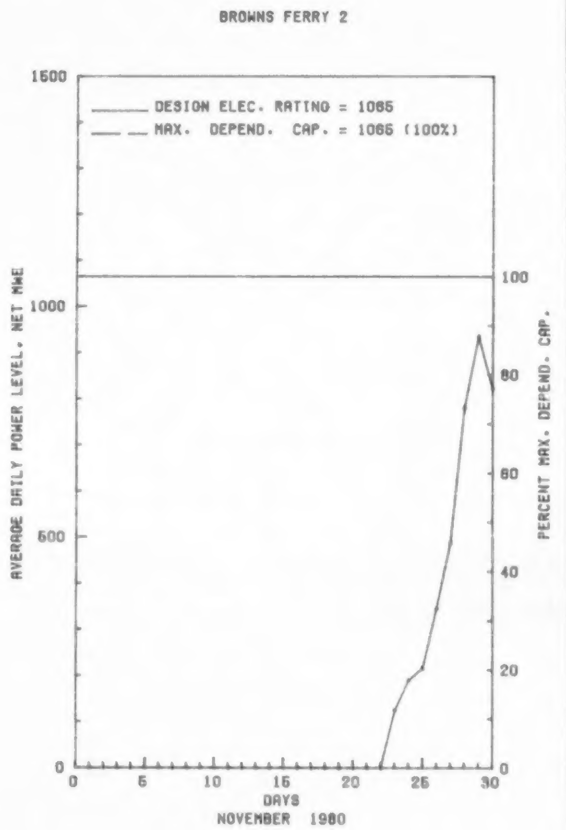
INSPECTION OCTOBER 14-15, (80-32): THIS SPECIAL, UNANNOUNCED INSPECTION INVOLVED THREE INSPECTOR-HOURS ON SITE IN THE AREA OF A LICENSEE IDENTIFIED ITEM CONCERNING SETTLING OF STRUCTURAL BACKFILL UNDER THE RHR SERVICE WATER TUNNELS. OF THE AREA INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATION WERE IDENTIFIED.

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 1-16-81

REVIEWED BY NRR DL: Dick Clark **DATE:** 01-19-81

BROWNS FERRY 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-260
2. Utility Contact: Don Green (205) 729-6846
3. Licensed Thermal Power (MWt): 3293
4. Nameplate Rating (Gross MWe): 1280 x 0.9 = 1152
5. Design Electrical Rating (Net MWe): 1065
6. Maximum Dependable Capacity (Gross MWe): 1098
7. Maximum Dependable Capacity (Net MWe): 1065
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	8,040.0	50,449.0
12. Hours Reactor Was Critical	230.0	5,534.1	30,018.5
13. Reactor Reserve Shutdown Hours	10.6	659.6	12,434.5
14. Hours Generator On-Line	184.2	5,355.3	29,019.8
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	300,518	15,230,393	81,385,393
17. Gross Electrical Energy (MWH)	96,710	5,058,720	26,998,788
18. Net Electrical Energy (MWH)	90,871	4,898,119	26,229,588
19. Unit Service Factor	25.6	66.6	57.5
20. Unit Availability Factor	25.6	66.6	57.5
21. Unit Capacity Factor (MDC Net)	11.9	57.2	48.8
22. Unit Capacity Factor (DER Net)	11.9	57.2	48.8
23. Unit Forced Outage Rate	.0	11.8	33.3

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
155	11/21/80	S	535.8	C	4				REFUELING CONCLUDED
156	11/30/80	S	.0	B	5				DERATED FOR RTI-23A (FEEDWATER SYSTEM)

SUMMARY

UNIT 2 CONCLUDED A REFUELING OUTAGE AND RESUMED POWER OPERATION. ON NOVEMBER 21, BEFORE THE UNIT WAS ON LINE, THE REACTOR SCRAMMED DURING STARTUP TESTING (MRTI) FOR BOC-4

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

best copy available

BROWNS FERRY 3

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Alabama/Limestone 10 miles NW of Decatur
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	3293
5.	Design Electrical Rating (MWE Net)	1065
6.	Date of Initial Criticality	August 8, 1976
7.	Date of Commercial Operation	March 1, 1977
8.	Condenser Cooling Method	Once Through With Forced Draft Cooling Tower
9.	Condenser Cooling Water Source	Tennessee River

Utility & Contractor Information

10.	Licensee	Tennessee Valley Authority
11.	Corporate Address	500A Chestnut Street Tower II Chattanooga, Tennessee 37401
12.	Corporate Contact	H. G. Parris, Manager of Power
13.	Architect/Engineer	TVA
14.	Nuclear Steam Supply System	GE
15.	Constructor	TVA

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R.C. Lewis
18.	IE Section Chief	H. C. Dance
19.	IE Resident Inspector	R. Sullivan (205) 729-6196
20.	Licensing Project Manager	R. Clark
21.	Docket Number	50-296
22.	License Number/Date of Issue	DPR-68, 7/2/76
23.	Public Document Room	Athens Public Library South and Forrest Athens, Alabama 35611

INSPECTION STATUS

ENFORCEMENT STATUS

AS REQUIRED BY TECHNICAL SPECIFICATION 3.11.E.2, AN INSPECTION AND AUDIT BY AN OUTSIDE QUALIFIED FIRE CONSULTANT WILL BE PERFORMED AT INTERVALS NO GREATER THAN 3 YEARS. CONTRARY TO THE ABOVE, ON SEPTEMBER 1, 1980, GREATER THAN THREE YEARS HAD ELASPED SINCE THE LAST FIRE INSPECTION AND AUDIT BY AN OUTSIDE AGENCY.
(8029 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

SHUTDOWN FOR REFUELING.

+ LAST IE SITE INSPECTION DATE: OCTOBER 14-15, 1980

+ INSPECTION REPORT NO: 50-296/80-32

SUMMARY

INSPECTION SUMMARY

+ INSPECTION SEPTEMBER 1-30 (80-35): THIS ROUTINE INSPECTION INVOLVED 38 RESIDENT INSPECTOR-HOURS ON SITE IN THE AREAS OF OPERATIONAL SAFETY, IE CIRCULAR FOLLOWUP, REPORTABLE OCCURRENCES, PLANT PHYSICAL PROTECTION, FIRE PROTECTION, REFUELING OUTAGE, REACTOR TRIP, IE BULLETIN FOLLOWUP, AND HEALTH PHYSICS. OF THE NINE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE FOUND IN SIX AREAS; THREE SEPARATE ITEMS OF NONCOMPLIANCE WERE FOUND IN THREE AREAS; (INFRACTION - FAILURE TO FOLLOW PROCEDURE FOR FUEL ASSEMBLY ORIENTATION VERIFICATION; INFRACTION - FAILURE TO HAVE AN OUTSIDE AGENCY PERFORM A FIRE INSPECTION WITHIN THREE YEARS; INFRACTION - FAILURE OF PERSONNEL TO HAVE A SPECIAL WORK PERMIT FOR ENTRY INTO A HIGH RADIATION AREA.

INSPECTION OCTOBER 6-7 (80-31): INCLUDED REVIEW OF LICENSEE IDENTIFIED ITEM OF NONCOMPLIANCE AND LICENSEE PROCEDURE FOR UNESCORTED ACCESS CLEARANCES. THE INSPECTOR ALSO REVIEWED THE LICENSEES INVESTIGATION OF A RECENT INTRUSION OF THE PROTECTED AREA AT BROWNS FERRY. THE INSPECTION INVOLVED THREE INSPECTOR-HOURS ON SITE BY ONE NRC INSPECTOR. THE INSPECTION WAS BEGUN DURING A REGULAR PERIOD. THE LICENSEE WAS FOUND TO BE IN COMPLIANCE WITH NRC REQUIREMENTS WITHIN THE AREAS EXAMINED DURING THE INSPECTION.

INSPECTION OCTOBER 14-15, (80-32): THIS SPECIAL, UNANNOUNCED INSPECTION INVOLVED THREE INSPECTOR-HOURS ON SITE IN THE AREA OF A LICENSEE IDENTIFIED ITEM CONCERNING SETTLING OF STRUCTURAL BACKFILL UNDER THE RHR SERVICE WATER TUNNELS. OF THE AREA INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATION WERE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

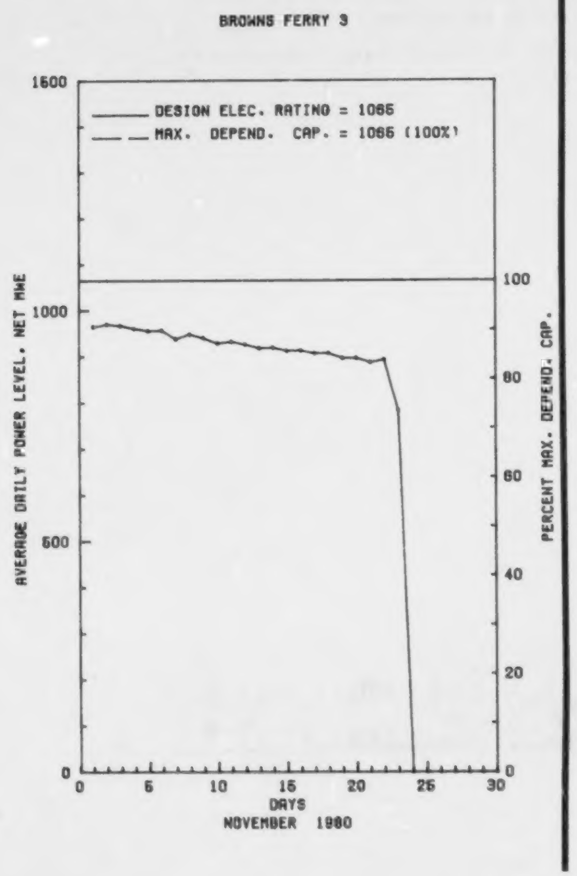
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE			

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 1-16-81

REVIEWED BY NRR DL: Dick Clark **DATE:** 01-19-81

BROWNS FERRY 3

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-296
2. Utility Contact: Don Green (205) 729-6846
3. Licensed Thermal Power (MWt): 3293
4. Nameplate Rating (Gross MWe): 1280 x 0.9 = 1152
5. Design Electrical Rating (Net MWe): 1065
6. Maximum Dependable Capacity (Gross MWe): 1098
7. Maximum Dependable Capacity (Net MWe): 1065
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	8,040.0	32,904.0
12. Hours Reactor Was Critical	549.7	7,063.9	25,971.6
13. Reactor Reserve Shutdown Hours	.0	152.6	1,811.0
14. Hours Generator On-Line	549.7	6,952.1	25,389.9
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,556,352	21,257,162	74,365,028
17. Gross Electrical Energy (MWH)	523,900	7,136,450	24,539,030
18. Net Electrical Energy (MWH)	507,548	6,936,550	23,824,323
19. Unit Service Factor	76.3	86.5	77.2
20. Unit Availability Factor	76.3	86.5	77.2
21. Unit Capacity Factor (MDC Net)	66.2	81.0	68.0
22. Unit Capacity Factor (DER Net)	66.2	81.0	68.0
23. Unit Forced Outage Rate	.0	11.7	9.8

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 01/08/81

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
153	11/23/80	S	170.3	C	1				EOC-3 REFUEL OUTAGE

SUMMARY

THE UNIT OPERATED AT EOC POWER. ON NOVEMBER 23, THE REACTOR WAS MANUALLY SCRAMMED TO BEGIN ITS SCHEDULED EOC-3 REFUELING OUTAGE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

BRUNSWICK 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	North Carolina/Brunswick 3 miles N of Southport
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2436
5.	Design Electrical Rating (MWE Net)	821
6.	Date of Initial Criticality	October 8, 1976
7.	Date of Commercial Operation	March 18, 1977
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Cape Fear River

Utility & Contractor Information

10.	Licensee	Carolina Power and Light Company
11.	Corporate Address	411 Fayetteville Street Raleigh, North Carolina 27602
12.	Corporate Contact	J. A. Jones Executive Vice President
13.	Architect/Engineer	United Engineers & Constructors, Inc.
14.	Nuclear Steam Supply System	GE
15.	Constructor	Brown and Root, Inc.

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R.C. Lewis
18.	IE Section Chief	P.J. Kellogg
19.	IE Resident Inspector	Don Johnson (919) 457-9531
20.	Licensing Project Manager	J. Hannon
21.	Docket Number	50-325
22.	License Number/Date of Issue	DPR-71, 9/8/76
23.	Public Document Room	Southport-Brunswick County Library 109 W. Moore Street Southport, North Carolina 28461

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

ROUTINE POWER OPERATION

- LAST IE SITE INSPECTION DATE: OCTOBER 20-31, 1980
- INSPECTION REPORT NO: 50-325/80-42

SUMMARY

INSPECTION SUMMARY

- NO INSPECTION REPORTS RELEASED THIS MONTH.

REPORTS RECEIVED FROM LICENSEE

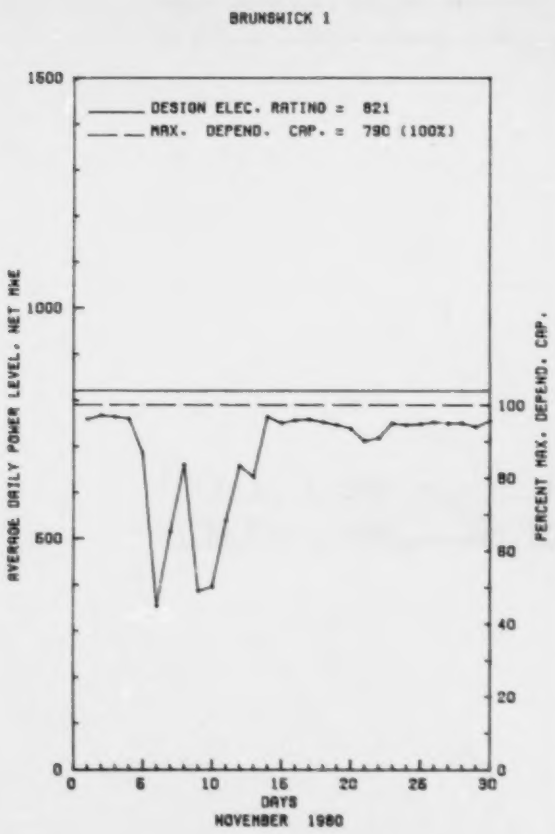
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-079/03L-0	10/09/80	11/07/80	REACTOR WATER LEVEL INSTRUMENT CHANNEL NO. 1 INOPERABLE
80-080/03L-0	09/23/80	10/23/80	A REACTOR LOW WATER LEVEL SWITCH FAILED TO ACTUATE DURING SURVEILLANCE
80-081/03L-0	10/10/80	11/07/80	NO. 3 DG FAILED TO START WITHIN REQUIRED 10 SECONDS
80-083/03L-0	09/26/80	10/24/80	CTMT ATMOSPHERIC OXYGEN ANALYZER FAILED TO LOSS OF SAMPLE FLOW.

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 1-16-81

REVIEWED BY NRR DL: John N. Hannon **DATE:** 1-19-81

BRUNSWICK 1

AVERAGE POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-325
2. Utility Contact: E. Willis (919) 457-9521
3. Licensed Thermal Power (MWt): 2436
4. Nameplate Rating (Gross MWe): 963 x 0.9 = 867
5. Design Electrical Rating (Net MWe): 821
6. Maximum Dependable Capacity (Gross MWe): 815
7. Maximum Dependable Capacity (Net MWe): 790
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	8,040.0	32,497.0
12. Hours Reactor Was Critical	720.0	5,749.6	23,139.2
13. Reactor Reserve Shutdown Hours	.0	.0	1,647.1
14. Hours Generator On-Line	720.0	5,386.2	21,719.0
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,349,226	10,867,255	45,696,751
17. Gross Electrical Energy (MWH)	509,576	3,625,420	14,864,239
18. Net Electrical Energy (MWH)	493,826	3,492,815	14,300,675
19. Unit Service Factor	100.0	67.0	66.8
20. Unit Availability Factor	100.0	67.0	66.8
21. Unit Capacity Factor (MDC Net)	86.8	55.0	55.7
22. Unit Capacity Factor (DER Net)	83.5	52.9	53.6
23. Unit Forced Outage Rate	.0	9.2	18.0

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
MAINTENANCE, 03/07/81, 1344 HOURS
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
025	11/05/80	S	13.9	H	5		ZZ	ZZZZZZ	REDUCED POWER FOR A ROD SEQUENCE EXCHANGE. ROD CHANGES ARE NECESSARY FOR AN EVEN FUEL BURNUP. NO CORRECTIVE ACTION REQUIRED.
026	11/08/80	F	41.7	B	5		HC	HTEXCH	REDUCED POWER TO LEAK CHECK 1B NORTH CONDENSER WATERBOX. LEAKING TUBES WERE LOCATED BY USING THE HELIUM LEAK CHECK METHOD. TWO TUBES WERE PLUGGED.

SUMMARY

BRUNSWICK 1 OPERATED WITH 1 REDUCTION FOR CONTROL ROD EXCHANGE AND 1 OTHER REDUCTION FOR EQUIPMENT FAILURE DURING NOVEMBER.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

BRUNSWICK 2

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	North Carolina/Brunswick 3 miles N of Southport
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2436
5.	Design Electrical Rating (MWE Net)	821
6.	Date of Initial Criticality	March 20, 1975
7.	Date of Commercial Operation	November 3, 1975
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Cape Fear River

Utility & Contractor Information

10.	Licensee	Carolina Power and Light Company
11.	Corporate Address	411 Fayetteville Street Raleigh, North Carolina 27602
12.	Corporate Contact	J. A. Jones Executive Vice President
13.	Architect/Engineer	United Engineers & Constructors, Inc.
14.	Nuclear Steam Supply System	GE
15.	Constructor	Brown and Root, Inc.

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R.C. Lewis
18.	IE Section Chief	P.J. Kellogg
19.	IE Resident Inspector	Don Johnson (919) 457-9531
20.	Licensing Project Manager	J. Hannon
21.	Docket Number	50-324
22.	License Number/Date of Issue	DPR-62, 12/27/74
23.	Public Document Room	Southport-Brunswick County Library 109 W. Moore Street Southport, North Carolina 28461

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

UNIT IN NORMAL OPERATION.

+ LAST IE SITE INSPECTION DATE: OCTOBER 20-31, 1980

+ INSPECTION REPORT NO: 50-324/80-39

SUMMARY

INSPECTION SUMMARY

+ NO INSPECTION REPORTS RELEASED THIS MONTH.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

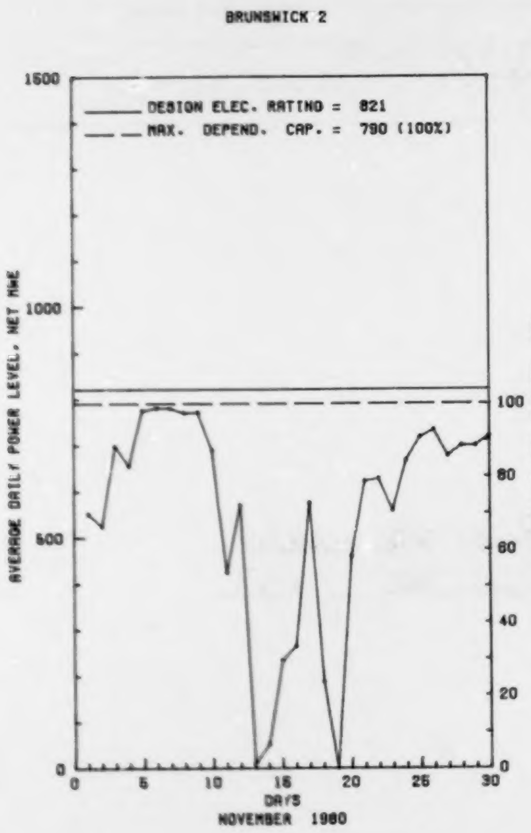
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-053/03L-0	10/08/80	11/07/80	REACTOR WATER LEVEL INSTRUMENT CHANNEL 1 OUT OF SERVICE
80-067/03L-0	09/23/80	10/22/80	RCIC INJECTION VALVE WOULD NOT OPEN FULLY DUE TO LACK OF TORQUE SWITCH JUMPER
80-073/03L-0	09/27/80	10/27/80	CTMT ATMOSPHERIC OXYGEN ANALYZER WOULD NOT OPERATE DUE TO LOW FLOW
80-074/03L-0	09/27/80	10/24/80	OXYGEN CONCENTRATION NOT REDUCED TO LESS THAN 4 PERCENT FOLLOWING INITIAL INERTING
80-079/03L-0	10/15/80	10/29/80	CONTAINMENT ATMOSPHERIC DILUTION SYSTEM WILL NOT MEET SINGLE FAILURE CRITERIA IN ONE RELAY

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 1-16-81

REVIEWED BY NRR DL: John N. Hannon **DATE:** 1-19-81

BRUNSWICK 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-324
2. Utility Contact: E. Willis (919) 457-9521
3. Licensed Thermal Power (MWt): 2436
4. Nameplate Rating (Gross MWe): 963 x 0.9 = 867
5. Design Electrical Rating (Net MWe): 821
6. Maximum Dependable Capacity (Gross MWe): 815
7. Maximum Dependable Capacity (Net MWe): 790
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	8,040.0	44,521.0
12. Hours Reactor Was Critical	683.6	3,221.4	28,545.3
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	635.6	2,709.8	26,583.8
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,262,123	5,278,925	49,013,387
17. Gross Electrical Energy (MWH)	410,737	1,757,617	16,372,691
18. Net Electrical Energy (MWH)	395,693	1,684,026	15,724,498
19. Unit Service Factor	88.3	33.7	59.7
20. Unit Availability Factor	88.3	33.7	59.7
21. Unit Capacity Factor (MDC Net)	69.6	26.5	44.7
22. Unit Capacity Factor (DER Net)	66.9	25.5	43.0
23. Unit Forced Outage Rate	11.7	13.3	13.0

24. Shutdowns Scheduled Over Next 6 Months (Type,Date,and Duration of Each): MAINTENANCE, 12/06/80, 171 HOURS
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
012	11/01/80	F	11.4	B	5		XX	PUMPXX	REDUCED POWER FOR HEATER DRAIN PUMP MAINTENANCE. REMOVED MOTOR FROM PUMP 2A AND PLACED ON PUMP 2B.
013	11/11/80	F	17.6	B	5		HC	HTEXCH	REDUCED POWER TO LEAK CHECK 2A SOUTH CONDENSER WATERBOX. LEAKING TUBES WERE LOCATED BY USING THE HELIUM LEAK CHECK METHOD. TWO TUBES WERE PLUGGED.
014	11/13/80	F	34.8	A	3		IA	ELECON	REACTOR SCRAM DUE TO A REACTOR PROTECTION SYSTEM POWER SUPPLY GROUP. INVESTIGATION REVEALED AN INSULATION BREAKDOWN AND BURN-THROUGH OF THE GROUND CABLE AT A POINT WHERE THE CABLE PHYSICALLY RESTED AGAINST A 120 VOLT CONTROL POWER RELAY CONTACT. CORRECTIVE ACTION CONSISTED OF REPAIRING THE DAMAGED INSULATION IN ACCORDANCE WITH PLANT PROCEDURES.
015	11/15/80	F	12.2	B	1		XX	PUMPXX	SEPARATED FROM GRID TO REPAIR LEAK IN HEATER DRAIN PUMPS. THE LEAK WAS ATTRIBUTED TO A FAILURE OF RECIRCULATION PIPING BETWEEN HEATER DRAIN PUMP 2C AND THE DEAERATOR TANK. THE 21" SECTION OF PIPE WAS CUT OUT AND A NEW SECTION WELDED IN PLACE.
016	11/18/80	F	37.3	A	3		HA	INSTRU	REACTOR SCRAM DUE TO A POWER LOAD UNBALANCE CONDITION SENSED BY THE TURBINE CONTROL EHC SYSTEM. REPLACED THE CONTROL INTERCEPT VALVE AMPLIFIER AND TRIGGER PRINTED WIRE BOARD.

SUMMARY

BRUNSWICK 2 OPERATED WITH 2 OUTAGES FOR EQUIPMENT FAILURE, 1 OUTAGE AND 2 REDUCTIONS FOR MAINTENANCE DURING NOVEMBER.

LEGEND

[1] Type:
- F–Forced
- S–Scheduled

[2] Reason:
- A–Equipment Failure
- B–Maintenance or Test
- C–Refueling
- D–Regulatory Restriction
- E–Operator Training & License Examination
- F–Administrative
- G–Operational Error
- H–Other

[3] Method:
- 1–Manual
- 2–Manual Scram
- 3–Automatic Scram
- 4–Continuations
- 5–Load Reduction
- 9–Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

CALVERT CLIFFS 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Maryland/Calvert 40 miles S of Annapolis
2.	Electric Reliability Council	Mid-Atlantic Area Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2700
5.	Design Electrical Rating (MWE Net)	845
6.	Date of Initial Criticality	October 7, 1974
7.	Date of Commercial Operation	May 8, 1975
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Chesapeake Bay

Utility & Contractor Information

10.	Licensee	Baltimore Gas and Electric Company
11.	Corporate Address	P. O. Box 1475 Baltimore, Maryland 21203
12.	Corporate Contact	A. E. Lundvall, Jr. Vice President-Supply
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Combustion Engineering
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Section Chief	E. C. McCabe, Jr.
19.	IE Resident Inspector	R. Architzel (301) 586-2627
20.	Licensing Project Manager	E. L. Conner, Jr.
21.	Docket Number	50-317
22.	License Number/Date of Issue	DPR-53, 7/31/74
23.	Public Document Room	Calvert County Library Prince Frederick, Maryland 20678

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO 10 CFR 50, APPENDIX B, CRITERION X, INSPECTION ACTIVITIES WERE NOT ADEQUATE TO ENSURE PROPER SPACING OF THE SPENT FUEL RACKS AND AN OUTDATED DRAWING WAS USED FOR INSTALLATION. THIS WAS IDENTIFIED SEPTEMBER 12, 1980 FOLLOWING INSTALLATION OF RACKS NOS. 1, 2 AND 3. (8815 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS:

- RESTART FROM REFUELING OUTAGE WEEK OF DECEMBER 22, 1980.
- LAST IE SITE INSPECTION DATE: 11/1-30/80
- INSPECTION REPORT NO: 50-317/80-22

SUMMARY

INSPECTION SUMMARY

- 50-317/80-15 - SEP 1-29: ROUTINE, ONSITE REGULAR AND BACKSHIFT INSPECTION BY THE RESIDENT INSPECTORS (40 HRS). AREAS INSPECTED INCLUDED THE CONTROL ROOM AND THE ACCESSIBLE PORTIONS OF THE AUXILIARY, TURBINE, SERVICE, AND INTAKE BUILDINGS; RADIATION PROTECTION; PHYSICAL SECURITY; FIRE PROTECTION; PLANT OPERATING RECORDS; MAINTENANCE ACTIVITIES; OFFSITE SAFETY REVIEW COMMITTEE ACTIVITIES; RECEIPT OF NEW FUEL, EMERGENCY DRILL (SEPTEMBER 16), SPENT FUEL POOL MODIFICATIONS, AND REPORTING TO THE NRC. ONE NONCOMPLIANCE WAS IDENTIFIED: FAILURE TO ADEQUATELY INSPECT SPENT FUEL RACK INSTALLATION.
- 50-317/80-18 - OCT 14-16: UNANNOUNCED ONSITE INVESTIGATION BY ONE REGION-BASED INVESTIGATOR (15 HRS) OF ALLEGATIONS RELATING TO THE PIPE SUPPORT ACTIVITIES PERFORMED TO COMPLY WITH IE BULLETIN 79-02. NONE OF THE ALLEGATIONS WERE SUBSTANTIATED.
- 50-317/80-20 - OCT 27-29: ROUTINE, ANNOUNCED, ONSITE INSPECTION BY A REGION-BASED INSPECTOR (20 HRS) TO INSPECT INSTALLED CLASS 1E ELECTRICAL EQUIPMENT IN ACCORDANCE WITH IE BULLETIN 79-01B. THE INSPECTOR PERFORMED AN INSTALLATION VERIFICATION OF COMPONENTS OF THE CONDENSATE AND FEEDWATER SYSTEM, AND COMPRESSED AIR SYSTEM. ONE ITEM OF NONCOMPLIANCE WAS IDENTIFIED: FAILURE TO USE WRITTEN MAINTENANCE PROCEDURES OR A MAINTENANCE REQUEST.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

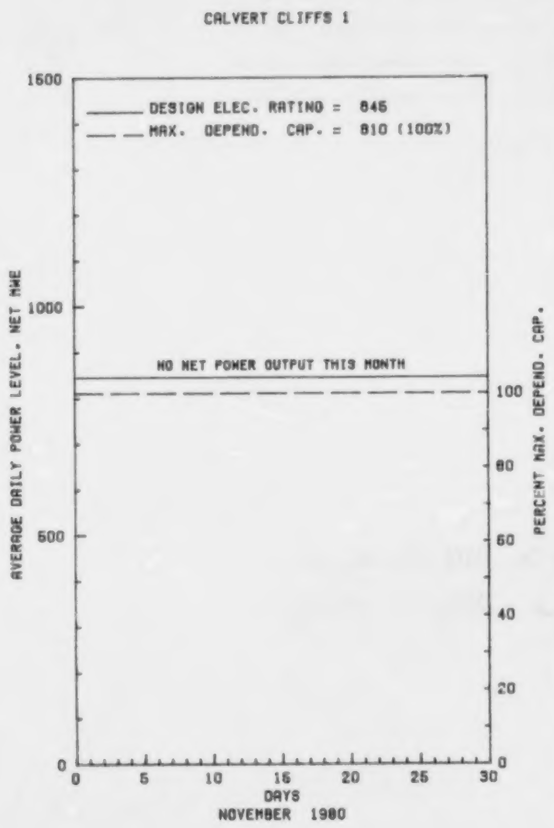
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-054/03L	09/25/80	10/25/80	ESFAS PRESSURIZER PRESSURE CHANNEL ZD INDICATED A DEVIATION OF GREATER THAN 0.16 VOLTS DURING NORMAL POWER OPERATION.
80-055/03L	10/01/80	10/31/80	ON OCT. 3 & 8 THE CTMT ATMOSPHERE PARTICULATE RMS DETECTOR FAILED.
80-057/03L	10/02/80	10/31/80	NO. 12 AFW PUMP TURBINE DISCOVERED TRIPPED. CAUSED BY PERSONS WORKING IN VICINITY.
80-059/03L	10/11/80	11/11/80	RCS LEAKAGE EXCEEDED T.S. LIMIT FOR UNIDENTIFIED (ACTUAL WAS 2.17 GPM) CAUSED BY PACKING LEAK ON PORV BLOCKING VALVE.
80-060/03L	10/18/80	11/17/80	RCS SAFETY VALVES RC-200 AND RC-201 FOUND WITH LIFT PRESSURE EXCEEDING MAXIMUM ALLOWABLE (70 AND 56 PSI, RESP.) DURING REFUELING OUTAGE SURVEILLANCE TESTING.

REVIEWED BY OMPA LOEB: MK Beebe **DATE:** 01/16/81

REVIEWED BY NRR DL: E L Conner **DATE:** 1/19/81

CALVERT CLIFFS 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-317
2. Utility Contact: S. D. Merson (301) 234-5248
3. Licensed Thermal Power (MWt): 2700
4. Nameplate Rating (Gross MWe): 1020 x 0.9 = 918
5. Design Electrical Rating (Net MWe): 845
6. Maximum Dependable Capacity (Gross MWe): 845
7. Maximum Dependable Capacity (Net MWe): 810
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	8,040.0	48,805.0
12. Hours Reactor Was Critical	.0	6,421.1	38,860.7
13. Reactor Reserve Shutdown Hours	.0	38.3	1,098.8
14. Hours Generator On-Line	.0	6,350.7	38,052.7
15. Unit Reserve Shutdown Hours	.0	0	.0
16. Gross Thermal Energy (MWH)	0	15,229,804	98,566,792
17. Gross Electrical Energy (MWH)	0	4,781,703	29,594,233
18. Net Electrical Energy (MWH)	-2,460	4,540,582	28,189,708
19. Unit Service Factor	.0	79.0	78.0
20. Unit Availability Factor	.0	79.0	78.0
21. Unit Capacity Factor (MDC Net)	.0	69.7	71.3
22. Unit Capacity Factor (DER Net)	.0	66.8	68.4
23. Unit Forced Outage Rate	.0	2.9	8.1

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 12/02/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-11	10/18/80	S	720.0	C	4		RC	FUELXX	REFUELING, UNIT GENERAL INSPECTION AND TMI MODIFICATIONS.

SUMMARY

THE UNIT CONTINUED REFUELING DURING THE MONTH OF NOVEMBER.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

CALVERT CLIFFS 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1. Location (state/county) (nearest population center)	Maryland/Calvert 40 miles S of Annapolis
2. Electric Reliability Council	Mid-Atlantic Area Council
3. Type of Reactor	PWR
4. Power Level (MWT)	2700
5. Design Electrical Rating (MWE Net)	845
6. Date of Initial Criticality	November 30, 1976
7. Date of Commercial Operation	April 1, 1977
8. Condenser Cooling Method	Once Through
9. Condenser Cooling Water Source	Chesapeake Bay

Utility & Contractor Information

10. Licensee	Baltimore Gas and Electric Company
11. Corporate Address	P. O. Box 1475 Baltimore, Maryland 21203
12. Corporate Contact	A. E. Lundvall, Jr. Vice President-Supply
13. Architect/Engineer	Bechtal
14. Nuclear Steam Supply System	Combustion Engineering
15. Constructor	Bechtel

Regulatory Information

16. IE Region responsible	I
17. IE Branch Chief	E. Brunner
18. I E Section Chief	E. C. MCCABE, JR.
19. IE Resident Inspector	R. Architzel (301) 586-2627
20. Licensing Project Manager	E. L. Conner, Jr.
21. Docket Number	50-318
22. License Number/Date of Issue	DPR-69, 8/13/76
23. Public Document Room	Calvert County Library Prince Frederick, Maryland 20678

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS:

+ REFUELING OUTAGE IS SCHEDULED TO COMMENCE JANUARY 1, 1981.

+ LAST IE SITE INSPECTION DATE: 11/1-30/80

+ INSPECTION REPORT NO: 50-318/80-18

SUMMARY

INSPECTION SUMMARY

+ 50-318/80-14 - SEP 1-29: ROUTINE, ONSITE REGULAR AND BACKSHIFT INSPECTION BY THE RESIDENT INSPECTORS (20 HRS). AREAS INSPECTED INCLUDED THE CONTROL ROOM AND THE ACCESSIBLE PORTIONS OF THE AUXILIARY, TURBINE, SERVICE, AND INTAKE BUILDINGS; RADIATION PROTECTION; PHYSICAL SECURITY; FIRE PROTECTION; PLANT OPERATING RECORDS; MAINTENANCE ACTIVITIES; OFFSITE SAFETY REVIEW COMMITTEE ACTIVITIES, RECEIPT OF NEW FUEL, EMERGENCY DRILL (SEPTEMBER 16), AND REPORTING TO THE NRC. NO NONCOMPLIANCES WERE IDENTIFIED.

+ 50-318/80-17 - OCT 27-29: ROUTINE, ANNOUNCED, ONSITE INSPECTION BY A REGION-BASED INSPECTOR (20 HRS) TO INSPECT INSTALLED CLASS 1E ELECTRICAL EQUIPMENT IN ACCORDANCE WITH IE BULLETIN 79-01B. THE INSPECTOR PERFORMED AN INSTALLATION VERIFICATION OF COMPONENTS OF THE CONDENSATE AND FEEDWATER SYSTEM, AND COMPRESSED AIR SYSTEM. NO NONCOMPLIANCES WERE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

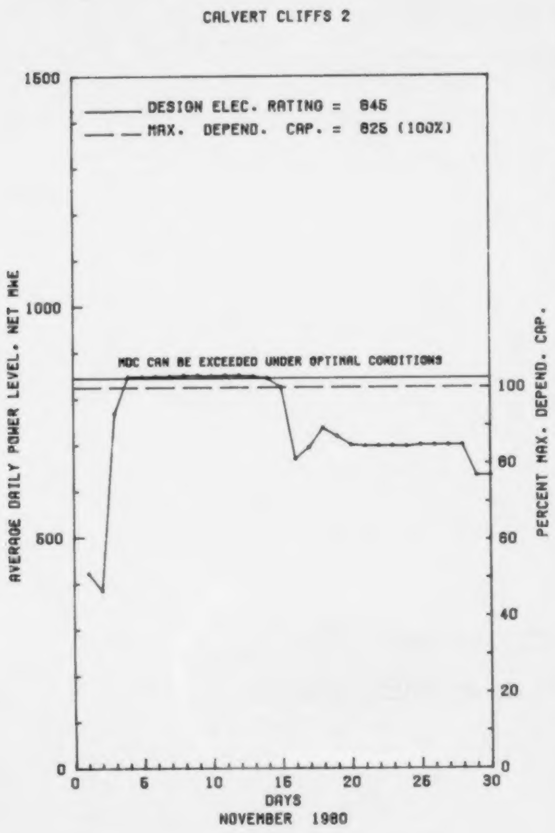
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-044/ 03L	09/26/80	10/24/80	ESFAS NO. 22 SGIS CHANNEL ZF INDICATED A DEVIATION OF GREATER THAN 0.1 VOLTS DURING NORMAL POWER OPERATION.
80-045/ 03L	10/17/80	11/17/80	CONTAINMENT PARTICULATE RADIOACTIVITY DETECTOR INOPERABLE CAUSE WAS BROKEN LEAD IN TERMINATION IN CONNECTOR
80-048/ 03L	10/19/80	11/19/80	CONTROL ELEMENT ASSEMBLIES 55 AND 61 DROPPED AT DIFFERENT TIMES WHILE CONTROLLING FLUX SHAPE
80-049/ 01T	11/06/80	11/20/80	PERIODIC VERIFICATION OF AC SOURCES NOT PERFORMED DURING PERIOD WHEN BOTH 4KV BUSES SUPPLIED BY 500KV RED BUS
80-050/ 01T	10/21/80	11/05/80	NO. 12 CONTROL ROOM AIR CONDITIONING UNIT TRIPPED WITH NO. 11 OOS

REVIEWED BY OMPA LOEB: AmR Beebe **DATE:** 01/16/81

REVIEWED BY NRR DL: E L Conner **DATE:** 1/19/81

CALVERT CLIFFS 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-318
2. Utility Contact: S. D. Merson (301) 234-5240
3. Licensed Thermal Power (MWt): 2700
4. Nameplate Rating (Gross MWe): 1012 x 0.9 = 911
5. Design Electrical Rating (Net MWe): 845
6. Maximum Dependable Capacity (Gross MWe): 860
7. Maximum Dependable Capacity (Net MWe): 825
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

ITEM 21 CALCULATED WITH A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	8,040.0	32,160.0
12. Hours Reactor Was Critical	720.0	7,730.9	27,675.8
13. Reactor Reserve Shutdown Hours	.0	28.7	441.8
14. Hours Generator On-Line	720.0	7,684.6	27,293.1
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,701,199	19,146,617	67,390,848
17. Gross Electrical Energy (MWH)	555,634	6,273,875	22,279,098
18. Net Electrical Energy (MWH)	530,057	5,988,931	21,245,951
19. Unit Service Factor	100.0	95.6	84.9
20. Unit Availability Factor	100.0	95.6	84.9
*21. Unit Capacity Factor (MDC Net)	89.2	90.3	80.1
22. Unit Capacity Factor (DER Net)	87.1	88.2	78.2
23. Unit Forced Outage Rate	.0	1.2	5.4

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
PLANNED OUTAGE (1/2-3/2/81), FOR REFUELING, GEN. INSPEC.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-15	11/01/80	S	56.8	H	5		RC	FUELXX	FUEL CONSERVATION, CAPACITY AVAILABLE ON DEMAND.

SUMMARY

THE UNIT OPERATED IN A FUEL CONSERVATION MODE DURING MOST OF THE MONTH.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

NRC POWER RESTRICTIONS: NONE

COOK 1

FACILITY DATA

Facility Description

1. Location (state/county) (nearest population center)	Michigan/Berrien 11 miles S of Benton Harbor
2. Electric Reliability Council	East Central Area Reliability Coordination Agreement
3. Type of Reactor	PWR
4. Power Level (MWT)	3250
5. Design Electrical Rating (MWE Net)	1054
6. Date of Initial Criticality	January 18, 1975
7. Date of Commercial Operation	August 27, 1975
8. Condenser Cooling Method	Once Through
9. Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10. Licensee	AEP Corporation
11. Corporate Address	Indiana & Michigan Electric Co. 2 Broadway New York, New York 10004
12. Corporate Contact	John Dolan Senior Executive V. P. Eng.
13. Architect/Engineer	AEP
14. Nuclear Steam Supply System	Westinghouse
15. Constructor	Indiana & Michigan Power Co.

Regulatory Information

16. IE Region responsible	III
17. IE Branch Chief	R. Heishman
18. IE Section Chief	D.C. Boyd
19. IE Resident Inspector	E. R. Swanson (616) 465-6775
20. Licensing Project Manager	S. Miner
21. Docket Number	50-315
22. License Number/Date of Issue	DPR-58 issued 10/25/74
• 23. Public Document Room	Maude Preston Palenske Memorial Library 500 Market Street St. Joseph, Michigan 49085

INSPECTION STATUS

ENFORCEMENT STATUS

```
    I N S P E C T I O N   S T A T U S

    OTHER ITEMS OF INTEREST

    PLANT STATUS

    OPERATING ROUTINELY.

LAST IE SITE INSPECTION DATE:  JUNE 16 THROUGH SEPTEMBER 27, 1980

        INSPECTION REPORT NO:  80-13

    SUMMARY

    INSPECTION SUMMARY

*   NO INSPECTIONS CONDUCTED.
```

REPORTS RECEIVED FROM LICENSEE

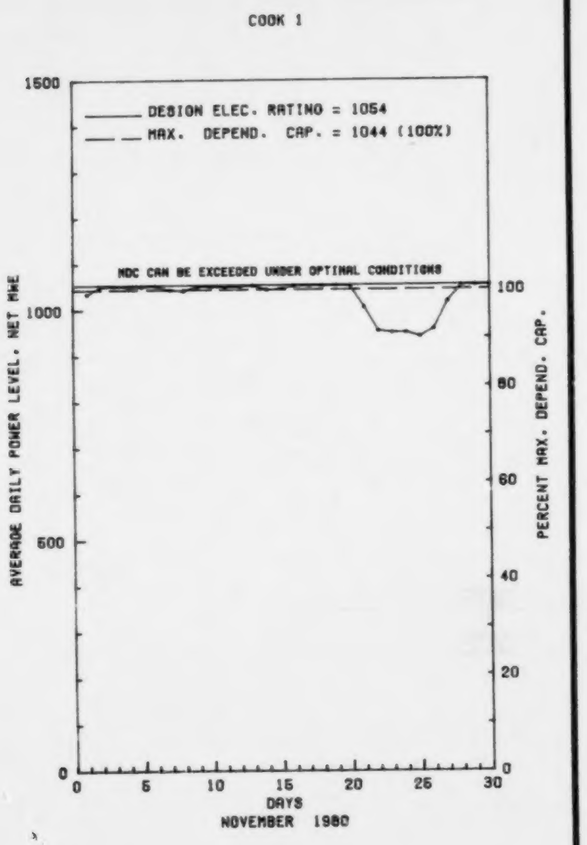
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
SI-14		11/17/80	A REACTOR TRIP OCCURRED DUE TO A FAILURE OF INVERTER CRID-IV FOR VITAL BUS CHANNEL IV. (SPECIAL REPORT)
80-25/ 03L-0	09/29/80	10/21/80	WHILE PERFORMING A SURVEILLANCE TEST ON CONTAINMENT ISOLATION VALVES, IT WAS DISCOVERED THAT ONE OF THE GLYCOL ISO. VALVES WOULD NOT FULLY CLOSE.

REVIEWED BY OMPA LOEB: P E Robe **DATE:** 1/16/81

REVIEWED BY NRR DL: Sydney Miner **DATE:** 1/16/81

COOK 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-315
2. Utility Contact: W. T. Gillett (616) 465-5901
3. Licensed Thermal Power (MWt): 3250
4. Nameplate Rating (Gross MWe): 1280 x 0.9 = 1152
5. Design Electrical Rating (Net MWe): 1054
6. Maximum Dependable Capacity (Gross MWe): 1080
7. Maximum Dependable Capacity (Net MWe): 1044
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	8,040.0	51,864.0
12. Hours Reactor Was Critical	720.0	5,992.3	38,954.0
13. Reactor Reserve Shutdown Hours	.0	.0	463.0
14. Hours Generator On-Line	720.0	5,907.0	37,990.2
15. Unit Reserve Shutdown Hours	.0	.0	321.0
16. Gross Thermal Energy (MWH)	2,293,722	18,326,826	107,393,131
17. Gross Electrical Energy (MWH)	768,160	6,084,250	35,241,590
18. Net Electrical Energy (MWH)	742,200	5,868,451	33,865,765
19. Unit Service Factor	100.0	73.5	76.5
20. Unit Availability Factor	100.0	73.5	76.5
21. Unit Capacity Factor (MDC Net)	98.7	69.9	67.5
22. Unit Capacity Factor (DER Net)	97.8	69.3	63.3
23. Unit Forced Outage Rate	.0	7.9	7.0

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
NONE									

SUMMARY

THERE WERE NO UNIT SHUTDOWNS OR SIGNIFICANT POWER REDUCTIONS DURING THE MONTH. THE UNIT OPERATED AT ESSENTIALLY 100% POWER WITH THE EXCEPTION OF A 10% POWER REDUCTION FROM 11/21/80 THROUGH 11/27/80 DUE TO A CIRCULATING WATER PUMP FAILURE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

COOK 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Michigan/Berrien 11 miles S of Benton Harbor
2.	Electric Reliability Council	East Central Area Reliability Coordination Agreement
3.	Type of Reactor	PWR
4.	Power Level (MWT)	3391
5.	Design Electrical Rating (MWE Net)	1100
6.	Date of Initial Criticality	March 10, 1978
7.	Date of Commercial Operation	July 1, 1978
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10.	Licensee	AEP Corporation Indiana & Michigan Electric Co.
11.	Corporate Address	2 Broadway New York, New York 10004
12.	Corporate Contact	John Dolan, VP/Chief Engineer (212) 422 - 4800
13.	Architect/Engineer	American Elec. Power Service Corp.
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Indiana & Michigan Power Co.

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	D.C. Boyd
19.	IE Resident Inspector	E. Swanson (616) 465-6775
20.	Licensing Project Manager	S. Miner
21.	Docket Number	50-316
22.	License Number/Date of Issue	DPR-74, 12/23/77
23.	Public Document Room	Maude Preston Palenske Memorial Library 500 Market Street St. Joseph, Michigan 49085

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

```
    I N S P E C T I O N   S T A T U S

    OTHER ITEMS OF INTEREST

    PLANT STATUS

    UNIT SHUTDOWN ON OCTOBER 18, 1980 DUE TO FAULT IN MAIN GENERATOR.  REPAIRS
    ESTIMATED TO REQUIRE 8 WEEKS.

LAST IE SITE INSPECTION DATE:  AUGUST 27-28, 1980

        INSPECTION REPORT NO:  80-12

    SUMMARY

    INSPECTION SUMMARY

+   NO INSPECTIONS CONDUCTED.
```

REPORTS RECEIVED FROM LICENSEE

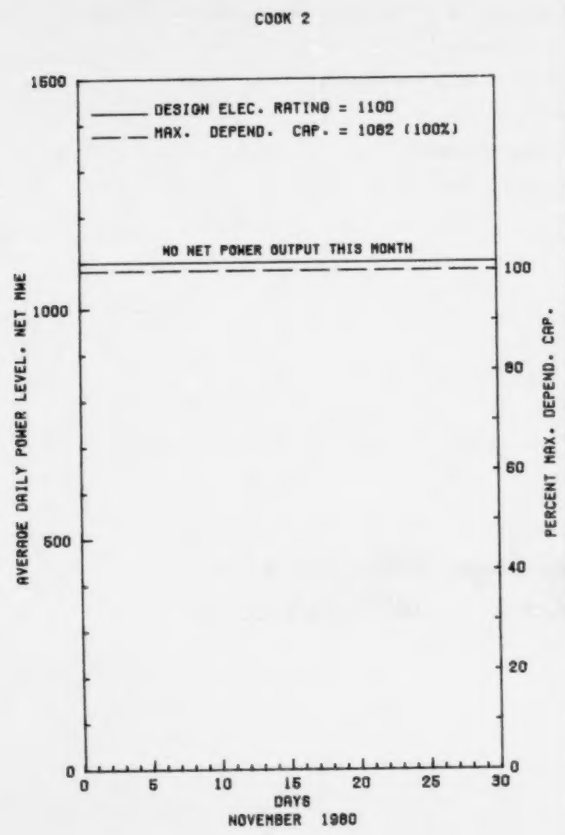
```
NUMBER   DATE OF   DATE OF   SUBJECT
         EVENT     REPORT

NONE.
===================================================================================================
```

REVIEWED BY OMPA LOEB: E Bobe **DATE:** 1/16/81

REVIEWED BY NRR DL: ____________ **DATE:** ____________

COOK 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-316
2. Utility Contact: W. T. Gillett (616) 465-5901
3. Licensed Thermal Power (MWt): 3391
4. Nameplate Rating (Gross MWe): 1333 x 0.85 = 1133
5. Design Electrical Rating (Net MWe): 1100
6. Maximum Dependable Capacity (Gross MWe): 1118
7. Maximum Dependable Capacity (Net MWe): 1082
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	8,040.0	25,560.0
12. Hours Reactor Was Critical	.0	6,160.4	17,250.3
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	.0	6,068.4	16,570.3
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	19,986,496	52,130,876
17. Gross Electrical Energy (MWH)	0	6,457,370	16,611,780
18. Net Electrical Energy (MWH)	0	6,228,971	15,996,383
19. Unit Service Factor	.0	75.5	71.9
20. Unit Availability Factor	.0	75.5	71.9
21. Unit Capacity Factor (MDC Net)	.0	71.6	66.7
22. Unit Capacity Factor (DER Net)	.0	70.4	66.0
23. Unit Forced Outage Rate	100.0	15.6	14.6

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 12/10/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
89	10/18/80	F	720.0	A	4		HA	GENERA	ELECTRICAL GENERATOR REPAIR OUTAGE CONTINUED FROM PREVIOUS MONTH. THE UNIT REMAINED OUT OF SERVICE AT THE END OF THE MONTH. RETURN TO SERVICE IS SCHEDULED FOR THE WEEK OF 12/08/80

SUMMARY

THE UNIT DID NOT OPERATE IN THE MONTH OF NOVEMBER.

LEGEND

[1] Type:
F-Forced
S-Scheduled

[2] Reason:
A-Equipment Failure
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error
H-Other

[3] Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Continuations
5-Load Reduction
9-Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

COOPER STATION

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Nebraska/Nemaha 23 miles S of Nebraska City
2.	Electric Reliability Council	Mid-Continent Area Reliability Coordination Agreement
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2381
5.	Design Electrical Rating (MWE Net)	778
6.	Date of Initial Criticality	February 21, 1974
7.	Date of Commercial Operation	July 1, 1974 (50% rating)
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Missouri River

Utility & Contractor Information

10.	Licensee	Nebraska Public Power District
11.	Corporate Address	P. O. Box 499 Columbus, Nebraska 68601
12.	Corporate Contact	J. M. Pilant, Director Licensing and Quality Assurance
13.	Architect/Engineer	Burns and Roe, Inc.
14.	Nuclear Steam Supply System	GE
15.	Constructor	Burns and Roe, Inc.

Regulatory Information

16.	IE Region responsible	IV
17.	IE Branch Chief	G. L. Madsen
18.	IE Section Chief	T. F. Westerman
19.	IE Resident Impector	R. G. Spangler
20.	Licensing Project Manager	V. L. Rooney
21.	Docket Number	50-298
22.	License Number/Date of Issue	DPR-46, 1/18/74
23.	Public Document Room	Auburn Public Library 118 15th Street Auburn, Nebraska 68305

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

FACILITY ITEMS (PLANS AND PROCEDURES):

MANAGERIAL ITEMS:

PLANT STATUS:

OPERATING AT REDUCED POWER.

LAST IE SITE INSPECTION DATE: OCTOBER 21-24, 1980 +

INSPECTION REPORT NO: 50-298/80-16 +

SUMMARY

INSPECTION SUMMARY

+ INSPECTION ON OCTOBER 21-24, AND 29-31, 1980 (80-16): ROUTINE, UNANNOUNCED INSPECTION OF FOLLOWUP TO PREVIOUSLY IDENTIFIED ITEMS, FOLLOWUP ON IEB'S AND FOLLOWUP TO SHORT TERM LESSONS LEARNED CATEGORY A ITEMS. NO ITEMS OF NONCOMPLIANCE OR DEVIATION WERE IDENTIFIED.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

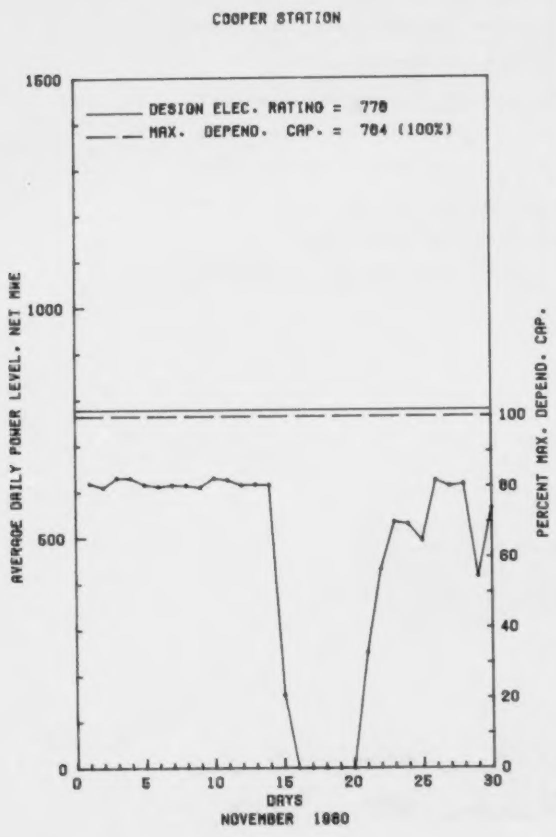
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-36 03L-0	10/3/80	10/31/80	FAILURE OF AN AIR OPERATED ISOLATION VALVE TO STROKE IN THE REQUIRED TIME
80-37 03L-0	10/6/80	11/3/80	FAILURE OF MO-89B RHR SERVICE WATER CONTROL VALVE TO FUNCTION
80-38 03L-0	10/11/80	11/10/80	REACTOR VESSEL LEVEL SWITCH FOUND OUT OF TECHNICAL SPECIFICATION LIMIT
80-39 03L-0	10/19/80	11/19/80	SNUBBER FOUND INOPERABLE
80-40 03L-0	10/17/80	11/7/80	FAILURE OF CONTROL SWITCH ON BREAKER IFS
80-41 03L-0	10/16/80	11/14/80	SW PUMP VIBRATING EXCESSIVELY
80-42 03L-0	10/20/80	11/18/80	SW-MO-89A FOUND INOPERABLE
80-43 03L-0	10/21/80	11/20/80	SW-MO-89B FAILED TO OPEN

REVIEWED BY OMPA LOEB: John J. Ruggio **DATE:** 1-16-81

REVIEWED BY NRR DL: VRooney **DATE:** 4/8/81

COOPER STATION

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-298
2. Utility Contact: P. J. Borer (402) 825-381:
3. Licensed Thermal Power (MWt): 2381
4. Nameplate Rating (Gross MWe): 983 x 0.85 = 836
5. Design Electrical Rating (Net MWe): 778
6. Maximum Dependable Capacity (Gross MWe): 787
7. Maximum Dependable Capacity (Net MWe): 764
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): 645
10. Reasons for Restrictions, If Any:
TEMPORARY TURBINE MODIFICATIONS

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	8,040.0	56,281.0
12. Hours Reactor Was Critical	598.4	5,608.1	46,845.4
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	582.8	5,498.2	45,973.8
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,241,664	11,879,136	88,061,862
17. Gross Electrical Energy (MWH)	346,095	3,456,355	27,995,203
18. Net Electrical Energy (MWH)	333,481	3,321,940	26,980,217
19. Unit Service Factor	80.9	68.4	81.7
20. Unit Availability Factor	80.9	68.4	81.7
21. Unit Capacity Factor (MDC Net)	60.6	54.1	62.7
22. Unit Capacity Factor (DER Net)	59.5	53.1	61.6
23. Unit Forced Outage Rate	7.6	2.4	4.4

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
REFUELING AND MAINTENANCE OUTAGE, MAY 3, 1980, 8 WEEKS
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-12	11/15/80	F	48.0	G	3	80-44	HE	INSTRU	DURING PERFORMANCE OF A SURVEILLANCE PROCEDURE CHECKING THE APRM TRIP SETTINGS, OPERATORS TRIPPED BOTH REACTOR PROTECTION TRIP CHANNELS CAUSING A REACTOR SCRAM. AN UNRELATED MALFUNCTION IN THE TURBINE CONTROL SYSTEM CAUSED THE MAIN STEAM BYPASS VALVES TO FAIL OPEN ALLOWING A RAPID VESSEL DEPRESSURIZATION AND COOLDOWN. A BAD TURBINE CONTROL COMPUTER CIRCUIT BOARD WAS REPLACED TO CORRECT THE MALFUNCTION.
	11/15/80	S	89.2	B	9				THE PLANT REMAINED SHUTDOWN FOR A SCHEDULED MAINTENANCE OUTAGE.
80-13	11/29/80	S	.0	H	5				REDUCED POWER TO ADJUST THE CONTROL ROD PATTERN.

SUMMARY

COOPER STATION OPERATED NORMALLY WITH 2 OUTAGES FOR OPERATIONAL ERROR AND MAINTENANCE, AND 1 REDUCTION FOR CONTROL ROD CHANGES.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

CRYSTAL RIVER 3

FACILITY DATA

Facility Description

1. Location (state/county) (nearest population center)	Florida/Citrus 7 miles NW of Crystal River
2. Electric Reliability Council	Southeastern Electric Reliability Council
3. Type of Reactor	PWR
4. Power Level (MWT)	2452
5. Design Electrical Rating (MWE Net)	825
6. Date of Initial Criticality	January 14, 1977
7. Date of Commercial Operation	March 13, 1977
8. Condenser Cooling Method	Once Through
9. Condenser Cooling Water Source	Gulf of Mexico

Utility & Contractor Information

10. Licensee	Florida Power Corporation
11. Corporate Address	3201 34th Street, South St. Petersbury, Florida 33733
12. Corporate Contact	J. Hancock
13. Architect/Engineer	Gilbert Associates, Inc.
14. Nuclear Steam Supply System	B&W
15. Constructor	J. A. Jones Construction Co.

Regulatory Information

16. IE Region responsible	II
17. IE Branch Chief	R. C. Lewis
18. IE Section Chief	R. Martin
19. IE Resident Inspector	T. F. Statka (904) 795-7677
20. Licensing Project Manager	P. B. Erickson
21. Docket Number	50-302
22. License Number/Date of Issue	DPR-72, 12/3/76
23. Public Document Room	Crystal River Public Library 668 N.W. First Crystal River, Florida 32639

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO THE REQUIREMENTS OF 10CFR50, APPENDIX B, CRITERION V, THE REQUIREMENTS OF SURVEILLANCE PROCEDURE SP-182 WERE NOT FOLLOWED IN PERFORMANCE OF THE REACTOR BUILDING TENDON SURVEILLANCE INSPECTION. (8036 INFRACTION)

CONTRARY TO CRITERION V OF 10CFR50 APPENDIX B, THE REQUIREMENTS OF SURVEILLANCE PROCEDURE SP-180 WERE NOT FOLLOWED IN PERFORMANCE OF THE REACTOR BUILDING DOME SURVEILLANCE. (8036 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

+ OPERATION AT FULL THERMAL POWER.

+ LAST IE SITE INSPECTION DATE: NOVEMBER 4-31, 1980

+ INSPECTION REPORT NO: 50-302/80-39

SUMMARY

INSPECTION SUMMARY

+ INSPECTION SEPTEMBER 2 - OCTOBER 3 (80-33): THIS ROUTINE INSPECTION BY THE RESIDENT INSPECTORS OF PLANT OPERATIONS, SECURITY, RADIOLOGICAL CONTROLS, LICENSEE EVENT REPORTS (LER'S), LICENSEE ACTION ON IE BULLETINS AND CIRCULARS, NON-ROUTINE EVENTS AND LICENSEE ACTION ON PREVIOUS INSPECTION ITEMS. FACILITY TOURS WERE CONDUCTED AND FACILITY OPERATIONS WERE OBSERVED. THE INSPECTION INVOLVED 115.5 HOURS ONSITE BY TWO RESIDENT INSPECTORS. THIS INSPECTION REPORT ALSO INCLUDES A MEETING WITH THE LOCAL PUBLIC OFFICIALS. THIS MEETING INVOLVED AN ADDITIONAL SIX HOURS OF INSPECTION EFFORT. ONE ITEM OF NONCOMPLIANCE WAS IDENTIFIED (DEFICIENCY - FAILURE TO MAKE PROPER TEMPORARY CHANGE TO PLANT REVIEW COMMITTEE (PRC) APPROVED PROCEDURE).

INSPECTION SEPTEMBER 16-19 (80-34): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 28 INSPECTOR-HOURS ON SITE IN THE AREAS OF MANAGEMENT CONTROLS; REVIEW OF PREVIOUS ENFORCEMENT MATTERS, REVIEW OF LICENSEE EVENT REPORTS; NONRAD WASTE RELEASES; CHEMICAL-INDUSTRIAL WASTE WATER TREATMENT SYSTEM. OF THE FIVE AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN FIVE AREAS.

INSPECTION SEPTEMBER 2 - OCTOBER 3 (80-33): THIS ROUTINE INSPECTION BY THE RESIDENT INSPECTORS OF PLANT OPERATIONS, SECURITY, RADIOLOGICAL CONTROLS, LICENSEE EVENT REPORTS (LER'S), LICENSEE ACTION ON IE BULLETINS AND CIRCULARS, NON-ROUTINE EVENTS AND LICENSEE ACTION ON PREVIOUS INSPECTION ITEMS. FACILITY TOURS WERE CONDUCTED AND FACILITY OPERATIONS WERE OBSERVED. THE INSPECTION INVOLVED 115.5 HOURS ON SITE BY TWO RESIDENT INSPECTORS. THIS INSPECTION REPORT ALSO INCLUDES A MEETING WITH THE LOCAL PUBLIC OFFICIALS. THIS MEETING INVOLVED AN ADDITIONAL 6 HOURS OF INSPECTION EFFORT. ONE ITEM OF NONCOMPLIANCE WAS IDENTIFIED (DEFICIENCY - FAILURE TO MAKE PROPER TEMPORARY CHANGE TO PLANT REVIEW COMMITTEE (PRC) APPROVED PROCEDURE).

INSPECTION SEPTEMBER 29 - OCTOBER 1 (80-36): THIS SPECIAL, ANNOUNCED INSPECTION INVOLVED 14 INSPECTOR-HOURS ON SITE IN THE AREAS OF FOLLOWUP ON IE BULLETIN 80-11, CONTAINMENT BUILDING DOME SURVEILLANCE PROGRAM, AND CONTAINMENT BUILDING TENDON SURVEILLANCE PROGRAM. OF THE THREE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN ONE AREA; TWO ITEMS OF NONCOMPLIANCE WAS FOUND IN TWO AREAS (INFRACTION - FAILURE TO PERFORM DOME SURVEILLANCE IN ACCORDANCE WITH THE SURVEILLANCE PROCEDURE; AND INFRACTION - FAILURE TO PERFORM THE TENDON SURVEILLANCE PROCEDURE IN ACCORDANCE WITH THE SURVEILLANCE PROCEDURE).

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

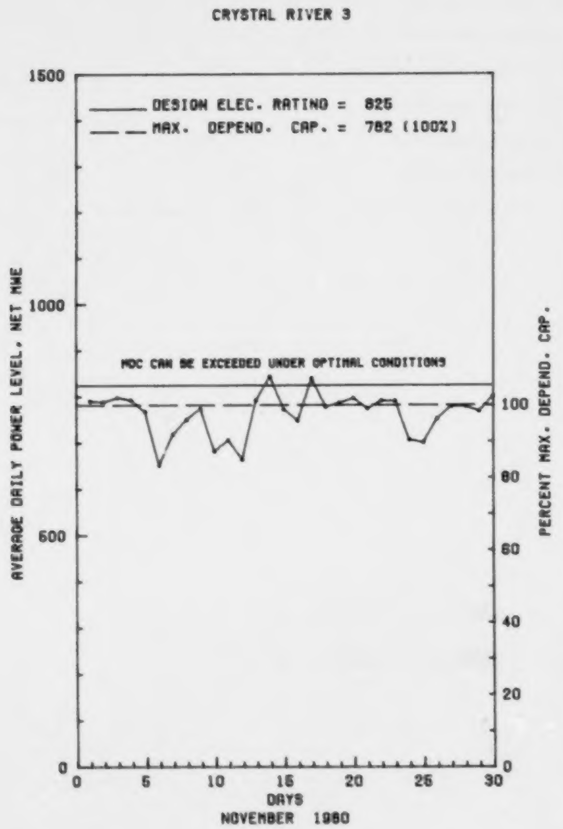
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-036/ 03L-0	08/27/80	09/24/80	"B" DECAY HEAT PUMP DISCHARGE THROTTLE VALVE WOULD NOT CONTROL FLOW IN AUTOMATIC
80-037/ 03L-0	09/03/80	10/01/80	A VALVE DHV110 IN THE DECAY HEAT REMOVAL SYSTEM WOULD NOT OPERATE ALSO AN INDICATOR LIGHT ON MUV23 FAILED
80-038/ 03L-0	09/16/80	10/14/80	"B" DECAY HEAT PUMP DISCHARGE THROTTLE VALVE WOULD NOT CONTROL FLOW IN AUTOMATIC
80-041/ 03L-0	10/02/80	10/29/80	RC DRAIN TANK ISOLATION WDV94 WOULD NOT CLOSE
80-042/ 03L-0	10/08/80	11/05/80	FOLLOWING A REACTOR STARTUP AFTER A TRIP THE DOSE EQUIVALENT I-131 SAMPLE EXCEEDED 1.0 MICROCURIE GRAM
80-043/ 03L-0	10/01/80	10/31/80	METABOLIC FUNCTION OF THE DISCHARGE SALTMARSHES (SPARTINA AND IUNCUS) SAMPLE OUT OF SPECTS
80-044/ 03L-0	09/05/80	11/05/80	REPORTS OF PH TEST RESULTS WERE NOT REPORTED CAUSING AN EVENT CONTRARY TO ETS
80-045/ 01T-0	10/28/80	11/07/80	CONTAINMENT PURGE ISOLATION VALVES INOPERABLE DUE TO RAD MONITOR NOT REPRESENTATIVELY SAMPLING
80-046/ 03L-0	10/16/80	11/13/80	EMERGENCY DIESEL GENERATOR EDG-1B WOULD NOT START
80-049/ 03L-0	09/23/80	10/16/80	FAILED TO VERIFY FIRE SUPPRESSION WATER SYSTEM OPERABLE AND VERIFY DELUGE AND SPRINKLER SYSTEM OPERABLE

REVIEWED BY OMPA LOEB: P E Kobe **DATE:** 1/16/81

REVIEWED BY NRR DL: P B Erickson **DATE:** 1/19/81

CRYSTAL RIVER 3

AVERAGE POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-302
2. Utility Contact: M.W. CULVER (904) 795-6486
3. Licensed Thermal Power (MWt): 2452
4. Nameplate Rating (Gross MWe): 989 x 0.9 = 890
5. Design Electrical Rating (Net MWe): 825
6. Maximum Dependable Capacity (Gross MWe): 821
7. Maximum Dependable Capacity (Net MWe): 782
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	8,040.0	32,616.0
12. Hours Reactor Was Critical	720.0	4,005.2	19,155.4
13. Reactor Reserve Shutdown Hours	.0	12.3	1,129.2
14. Hours Generator On-Line	720.0	3,940.6	18,640.9
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,678,649	8,734,293	41,051,284
17. Gross Electrical Energy (MWH)	576,568	2,972,468	13,944,236
18. Net Electrical Energy (MWH)	548,572	2,822,080	13,213,640
19. Unit Service Factor	100.0	49.0	57.2
20. Unit Availability Factor	100.0	49.0	57.2
21. Unit Capacity Factor (MDC Net)	97.4	44.9	51.8
22. Unit Capacity Factor (DER Net)	92.4	42.5	49.1
23. Unit Forced Outage Rate	.0	7.6	28.9

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-26	11/05/80	S	17.7	A	5		HC	HTEXCH	REDUCED POWER TO CLEAN "D" WATERBOX AND LOOK FOR LEAK.
80-27	11/07/80	F	16.0	A	5		HC	HTEXCH	REDUCED POWER FOR INCREASE IN "D" WATERBOX LEAKAGE. LEAKS WERE FOUND AND PLUGGED.
80-28	11/09/80	F	2.3	B	5		HC	HTEXCH	REDUCED POWER TO CHECK FOR SOURCE OF NOISE IN "D" WATERBOX. SACRIFICIAL METAL ON THE INLET SIDE NEEDED TIGHTENING.
80-29	11/10/80	S	10.8	B	5		HC	HTEXCH	REDUCED POWER TO CLEAN "A" WATERBOX.
80-30	11/10/80	S	9.8	B	5		HC	HTEXCH	REDUCED POWER TO CLEAN "B" WATERBOX.
80-31	11/11/80	S	8.6	B	5		HC	HTEXCH	REDUCED POWER TO CLEAN "C" WATERBOX.
80-32	11/12/80	S	9.0	A	5		HC	HTEXCH	REDUCED POWER TO FIND A SALTWATER LEAK IN "D" WATERBOX.
80-33	11/15/80	S	17.8	B	5		HB	ZZZZZZ	VARYING POWER BETWEEN 95 AND 100% FP DURING TESTING TO DETERMINE WHERE UNIT EFFICIENCY CAN BE IMPROVED AND MORE MWE GAINED.
80-34	11/24/80	F	12.1	A	5		HC	HTEXCH	REDUCED POWER TO LOCATE A SALTWATER LEAK IN "D" WATERBOX. LEAKING TUBES WERE LOCATED AND PLUGGED.
80-35	11/25/80	F	.4	A	5		HH	CKTBRK	REDUCED POWER DUE TO A FAILURE IN THE MAIN CONTROLLER FOR THE MAGNETIC COUPLING ON A CONDENSATE PUMP. COUPLING ON A CONDENSATE PUMP. COUPLING WAS SWITCHED TO THE AUXILIARY CONTROLLER AND THE PUMP RESTORED TO SERVICE.
80-36	11/26/80	F	17.2	A	5		HB	HTEXCH	POWER WAS SLIGHTLY REDUCED WHEN "A" AND "D" MOISTURE SEPARATOR/REHEATERS WERE ISOLATED DUE TO SUSPECTED TUBE LEAKS.
80-37	11/26/80	S	2.0	H	5		HH	CKTBRK	REDUCED POWER TO PLACE THE COUPLING FOR THE CONDENSATE PUMP BACK ON THE MAIN CONTROLLER.
80-38	11/28/80	S	.8	F	5		ZZ	ZZZZZZ	HELD AT SLIGHTLY REDUCED POWER BY DISPATCHER.

SUMMARY

CRYSTAL RIVER 3 OPERATED WITH 13 REDUCTIONS LISTED IN DETAIL ABOVE DURING NOVEMBER.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

NRC POWER RESTRICTIONS: NONE

DAVIS-BESSE 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Ohio/Ottawa 21 miles E of Toledo
2.	Electric Reliability Council	East Central Area Reliability Coordination Agreement
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2772
5.	Design Electrical Rating (MWE Net)	906
6.	Date of Initial Criticality	August 12, 1977
7.	Date of Commercial Operation	11/21/77 (25%); 12/19/77 (40%) 01/23/78 (75%); 07/31/78 (100%)
8.	Condenser Cooling Method	Cooling Tower
9.	Condenser Cooling Water Source	Lake Erie

Utility & Contractor Information

10.	Licensee	Toledo Edison Co.
11.	Corporate Address	Edison Plaza, 300 Madison Avenue Toledo, Ohio 43652
12.	Corporate Contact	R. P. Crouse, V. P., Nuclear (419)259-5221
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	B&W
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	R. F. Warnick
19.	IE Resident Inspector	L. Reyes (419) 898-2775
20.	Licensing Project Manager	D. Garner
21.	Docket Number	50-346
22.	License Number/Date of Issue	NPF-3, 4/22/77
23.	Public Document Room	Ida Rupp Public Library 310 Madison Street Port Clinton, OH 43452

INSPECTION STATUS

ENFORCEMENT STATUS

10 CFR 50, APPENDIX B, SECTION VI REQUIRES THAT MEASURES ESTABLISHED TO CONTROL THE ISSUANCE OF DOCUMENTS SHALL ASSURE THAT DOCUMENTS, SUCH AS PROCEDURES, ARE USED AT THE LOCATION WHERE THE PRESCRIBED ACTIVITY IS PERFORMED. T.S. 6.8.1 REQUIRES THAT THE APPLICABLE PROCEDURES RECOMMENDED IN APPENDIX A OF REGULATORY GUIDE 1.33, NOVEMBER 1972 SHALL BE IMPLEMENTED. APPENDIX A OF REGULATORY GUIDE 1.33, NOVEMBER 1972 INCLUDES PROCEDURES FOR STARTUP, OPERATION AND SHUTDOWN OF SAFETY RELATED SYSTEMS. THE MAKEUP AND PURIFICATION SYSTEM IS A SAFETY RELATED SYSTEM. ON JULY 3, 1980, REVISION 10 OF PROCEDURE SP 1104.02, "MAKEUP AND PURIFICATION SYSTEM," WAS APPROVED AND DISTRIBUTED. CONTRARY TO THE ABOVE, ON SEPTEMBER 17, 1980, THE INSPECTOR OBSERVED THAT THE PROCEDURE BEING USED BY THE CONTROL ROOM OPERATORS TO ALIGN THE MAKEUP AND PURIFICATION SYSTEM WAS REVISION 9, NOT THE LATEST APPROVED REVISION (REVISION 10).
(8025 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

FACILITY ITEMS

+ THE OUTAGE SCHEDULED HAS BEEN EXTENDED FOR APPROXIMATELY TWO WEEKS TO PERFORM REPAIRS ON ONE OF THE RCS TO LPI ISOLATION CHECK VALVES. TWO BOLTS AND THEIR LOCKING MECHANISM FOR THE CHECK VALVE WERE MISSING FROM THE VALVE. AN EVALUATION FOR OPERATION OF THE UNIT WITH THE LOOSE PARTS HAS BEEN PREPARED.

PLANT STATUS

+ THE REFUELING/MAINTENANCE OUTAGE WAS CONCLUDED ON NOVEMBER 1, 1980. ROUTINE COMMERCIAL OPERATION AT MONTHS END.

+LAST IE SITE INSPECTION DATE: SEPTEMBER 22 THROUGH OCTOBER 17, 1980

+ INSPECTION REPORT NO: 80-27

SUMMARY

INSPECTION SUMMARY

+ INSPECTION ON JUNE 9-11 AND OCTOBER 20-24, (80-16): ROUTINE, UNANNOUNCED INSPECTION OF MAINTENANCE DURING REFUELING OUTAGE, REVIEW OF PLANT OPERATIONS, CRYSTAL RIVER III LESSONS LEARNED AND EQUIPMENT CALIBRATIONS. THE INSPECTION INVOLVED 74 INSPECTOR-HOURS ONSITE BY TWO NRC INSPECTORS INCLUDING 6 INSPECTOR-HOURS ONSTIE DURING OFF-SHIFTS. OF THE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

+ INSPECTION ON SEPTEMBER 22-26, 29-30; OCTOBER 1-3, 6-10 AND 14-17, (80-27): ROUTINE RESIDENT INSPECTION OF FOLLOWUP ON PREVIOUS INSPECTION FINDINGS, MONTHLY MAINTENANCE OBSERVATION, MONTHLY SURVEILLANCE OBSERVATION, FOLLOWUP OF LICENSEE EVENT REPORTS, FOLLOWUP OF IE BULLETINS, INSPECTION DURING LONG TERM SHUTDOWN, REVIEW OF PLANT OPERATIONS DURING REFUELING OUTAGE, INDEPENDENT INSPECTION EFFORT (ALLEGATION ON WELDER QUALIFICATION, ALLEGATION ON FIRE RETARDANT WORK COMPLETION, TESTING OF DECAY HEAT REMOVAL ISOLATION VALVES), EMERGENCY PLANNING AND ACTIONS TAKEN ON TMI LESSONS LEARNED. THE INSPECTION INVOLVED 119 INSPECTOR-HOURS ONSITE BY TWO NRC INSPECTORS INCLUDING 26 INSPECTOR-HOURS ONSITE DURING OFF-SHIFTS. NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

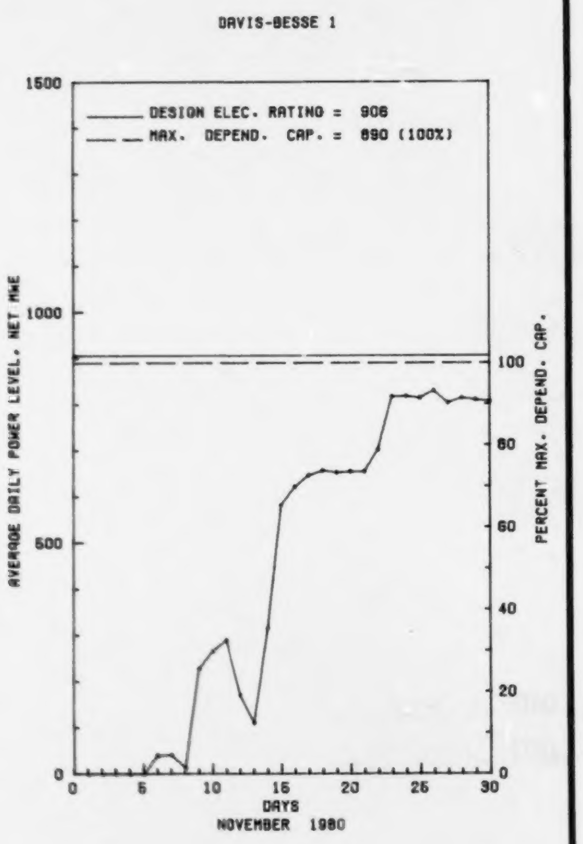
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-71/ 03L-0	09/23/80	10/21/80	EMERGENCY DIESEL GENERATOR TURBO-CHARGER FAILURE.
80-72/ 03L-0	10/02/80 UPDATED 10/31/80	10/31/80	FAILURE OF CV5011E, AIR SAMPLE RETURN INLET VALVE TO CONTAINMENT.
80-73/ 03L-0	10/08/80	11/05/80	INNER DOOR OF CONTAINMENT PERSONNEL LOCK COULD NOT BE LATCHED.
80-74/ 03L-0	10/08/80	11/05/80	CONTAINMENT AIR LOCK TEST HAS BEEN EXCEEDED.
80-75/ 03L-0	10/08/80	11/05/80	LEVEL INDICATION FOR SFRCS SG LEVEL INSTRUMENT READING HIGH.
80-76/ 03L-0	10/08/80	10/08/80	DISC UNIT DISENGAGED FROM THE BODY BLOCK DURING CORE FLOOD CHECK VALVE TEST.
80-77/ 03L-0	10/13/80	11/07/80	NINE OF TWENTY CONTROL ROOM NORMAL VENTILATION DAMPERS WOULD NOT CLOSE.

REVIEWED BY OMPA LOEB: PE Bobe **DATE:** 1/16/81

REVIEWED BY NRR DL: D J Garner **DATE:** 1/19/81

DAVIS-BESSE 1

AVERAGE POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-346
2. Utility Contact: BILAL SARSOUR (419) 259-5000 X[illegible]36
3. Licensed Thermal Power (MWt): 2772
4. Nameplate Rating (Gross MWe): 1069 x 0.9 = 962
5. Design Electrical Rating (Net MWe): 906
6. Maximum Dependable Capacity (Gross MWe): 934
7. Maximum Dependable Capacity (Net MWe): 890
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	8,040.0	28,565.0
12. Hours Reactor Was Critical	684.7	2,762.9	13,727.1
13. Reactor Reserve Shutdown Hours	6.3	6.3	2,882.1
14. Hours Generator On-Line	530.8	2,539.5	12,414.2
15. Unit Reserve Shutdown Hours	3.4	3.4	1,731.6
16. Gross Thermal Energy (MWH)	1,012,290	5,699,594	25,899,101
17. Gross Electrical Energy (MWH)	339,042	1,922,601	8,446,112
18. Net Electrical Energy (MWH)	309,064	1,792,851	7,963,429
19. Unit Service Factor	73.7	31.6	44.1
20. Unit Availability Factor	74.2	31.6	50.6
21. Unit Capacity Factor (MDC Net)	48.2	25.1	33.3
22. Unit Capacity Factor (DER Net)	47.4	24.6	32.7
23. Unit Forced Outage Rate	10.7	13.6	25.0

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
4	04/07/80	S	122.2	C	4				THE UNIT OUTAGE WHICH BEGAN ON APRIL 7, 1980 WAS STILL IN PROGRESS AT THE BEGINNING OF THE MONTH.
5	11/06/80	F	.7	B	1		XX	INSTRU	GENERATOR AUTOMATIC VOLTAGE CONTROL PROBLEMS.
6	11/06/80	F	22.5	A	3		XX	TRANSF	TURBINE/REACTOR TRIP DUE TO AN INDICATED FAULT ON THE AUXILIARY TRANSFORMER.
7	11/07/80	S	3.4	B	9				OVERSPEED TURBINE TRIP TEST.
8	11/08/80	F	19.4	A	3		XX	INSTRU	REACTOR TRIP ON HIGH REACTOR COOLANT SYSTEM PRESSURE DUE TO FAULTY HEADER PRESSURE ERROR SIGNAL COMPARATOR IN THE INTEGRATED CONTROL SYSTEM.
9	11/12/80	F	21.0	G	3	80-105			REACTOR TRIP ON HIGH REACTOR COOLANT SYSTEM PRESSURE.

SUMMARY

DAVIS BESSE RETURNED ON-LINE FROM A REFUELING MAINTENANCE OUTAGE ON THE 6TH, WITH 5 ADDITIONAL OUTAGES FOR MAINTENANCE, TESTING, EQUIPMENT FAILURE AND OPERATIONAL ERROR.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

NRC POWER RESTRICTIONS: NONE

DRESDEN 1

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Illinois/Grundy 9 miles E of Morris
2.	Electric Reliability Council	Mid-America Interpool Network
3.	Type of Reactor	BWR
4.	Power Level (MWT)	700
5.	Design Electrical Rating (MWE Net)	200
6.	Date of Initial Criticality	October 15, 1959
7.	Date of Commercial Operation	July 4, 1960
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Kankakee River

Utility & Contractor Information

10.	Licensee	Commonwealth Edison Co.
11.	Corporate Address	P. O. Box 767 Chicago, Illinois 60690
12.	Corporate Contact	C. Reed
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	GE
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	R. Spessard
19.	IE Resident Inspector	T. Tongue (815) 942-9267
20.	Licensing Project Manager	Paul W. O'Connor
21.	Docket Number	50-010
22.	License Number/Date of Issue	DPR-2 issued 9/28/59
23.	Public Document Room	Morris Public Library 604 Liberty Street Morris, Illinois 60451

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

THE LICENSEE DID NOT CONDUCT PROPER BACKGROUND SCREENING OF CONTRACTOR PERSONNEL.
(8016 DEFICIENCY)

10 CFR 50, APPENDIX B, CRITERION II REQUIRES ACTIVITIES AFFECTING QUALITY BE ACCOMPLISHED UNDER SUITABLY CONTROLLED CONDITIONS, INCLUDING ADEQUATE CLEANNESS. THE LICENSEE'S QUALITY ASSURANCE PROGRAM, SECTION 2.2 REQUIRES THAT THE LICENSEE ADHERE TO ALL MANDATORY REQUIREMENTS OF ANSI N18.7. ANSI N18.7-1976, SECTION 5.2.10 REQUIRES QUALITY HOUSEKEEPING PRACTICES ENCOMPASSING ALL ACTIVITIES RELATED TO CONTROL OF FIRE PREVENTION AND PROTECTION, INCLUDING DISPOSAL OF COMBUSTIBLE MATERIAL AND DEBRIS. CONTRARY TO THE ABOVE, DURING ROUTINE PLANT TOURS ON SEPTEMBER 19, 26, AND OCTOBER 3, 1980, THE INSPECTORS IDENTIFIED SEVERAL AREAS OF ACCUMULATED COMBUSTIBLE MATERIAL AND DEBRIS WHICH WERE NOT BEING CONTROLLED AND WHICH REPRESENTED A POTENTIAL FIRE HAZARD. EXAMPLES INCLUDED: SEPTEMBER 19, 1980, UNIT 1 TURBINE BUILDING TRACK WAY AND TURBINE FLOOR - PRESENCE OF PAPER, WOOD, PLASTIC AND SLINGS ON THE FLOOR AND UNIT 2 REACTOR BUILDING, R.W.C.U. DEMINERALIZER RECIRCULATION PUMP ROOM - PRESENCE OF MOP HEADS, RAGS, PAPER ON THE FLOOR AND GENERALLY DIRTY; ON SEPTEMBER 26, 1980, UNIT 3 REACTOR BUILDING, GROUND FLOOR WEST - PRESENCE OF EXCESSIVE SWP SHOE COVERS, WIPES, SCRAPS OF PAPER, PLASTIC AND GENERALLY DIRTY; AND ON OCTOBER 3, 1980, UNIT 2 LPCI ROOMS - PRESENCE OF USED SWP CLOTHING, PLASTIC BAGS, WIPES, AND A BAG OF TRASH AND UNIT 2 TORUS BASEMENT - PRESENCE OF HOSES, WIRES, CABLES, WOOD, PAPER, PLASTIC, DEBRIS AND RUBBLE IN LARGE QUANTITIES SHOWING EVIDENCE OF HAVING BEEN THERE FOR A LONG TIME. THIS IS A REPETITIVE ITEM OF NONCOMPLIANCE SINCE A SIMILAR PROBLEM WAS IDENTIFIED ONCE PREVIOUSLY IN NRC INSPECTION REPORT NO. 50-10/80-09; 50-237/80-08; 50-249/80-12, DATED MAY 23, 1980.
(8019 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

MANAGERIAL ITEMS

- EFFECTIVE AUGUST 14, 1980, MR. D. J. SCOTT BECAME STATION SUPERINTENDENT AT THE DRESDEN GENERATING STATION. (80-17)

PLANT STATUS

UNIT 1 SHUTDOWN ON OCTOBER 31, 1978 FOR AN EXTENDED OUTAGE TO UPGRADE ECCS, CHEMICAL CLEANING AND REFUELING. OUTAGE EXPECTED TO LAST THROUGH MID-1976.

- LAST IE SITE INSPECTION DATE: OCTOBER 31, 1980
- INSPECTION REPORT NO: 80-20

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE.			

SUMMARY

INSPECTION SUMMARY

- INSPECTION ON OCTOBER 8-10, (80-17): FACILITY ORGANIZATION AND OPERATION, MEASUREMENT AND CONTROLS, STORAGE AND INTERNAL CONTROLS, SHIPPING AND RECEIVING, INVENTORY, RECORDS AND REPORTS, AND MANAGEMENT OF MATERIAL CONTROL SYSTEM. THE INSPECTION INVOLVED 40 INSPECTOR-HOURS ONSITE BY TWO NRC INSPECTORS AND WAS BEGUN DURING THE REGULAR HOURS. THE LICENSEE WAS FOUND TO BE IN COMPLIANCE WITH NRC REQUIREMENTS IN THE SEVEN AREAS EXAMINED DURING THE INSPECTION.
- INSPECTION ON SEPTEMBER 7 THROUGH OCTOBER 3, (80-19): ROUTINE, UNANNOUNCED RESIDENT INSPECTION OF FOLLOWUP ON PREVIOUS NONCOMPLIANCE; MONTHLY SURVEILLANCE OBSERVATION; MONTHLY MAINTENANCE OBSERVATION; INSPECTION DURING LONG-TERM SHUT DOWN; IE BULLETIN 80-06 FOLLOWUP; AND, FOLLOWUP ON HEADQUARTERS REQUEST. THE INSPECTION INVOLVED 83 INSPECTOR-HOURS ONSITE BY TWO NRC INSPECTORS INCLUDING 15 HOURS ONSITE DURING OFF-SHIFTS. OF THE SEVEN AREAS INSPECTED, THERE WERE NO ITEMS OF NONCOMPLIANCE IN SIX AREAS. THERE WAS AN APPARENT ITEM OF NONCOMPLIANCE (FAILURE TO ADHERE TO QA PROGRAM REQUIREMENTS) IN ONE AREA.
- MANAGEMENT MEETING ON OCTOBER 31, (80-20): MANAGEMENT MEETING HELD AT THE NRC'S REQUEST TO DISCUSS THE REGULATORY PERFORMANCE OF THE ACTIVITIES AT DRESDEN, QUAD-CITIES, AND ZION NUCLEAR GENERATING STATIONS AS CONCLUDED IN THE SYSTEMATIC ASSESSMENT OF LICENSEE PERFORMANCE (SALP) PROGRAM. THE MEETING INVOLVED 18 MAN-HOURS AT THE CORPORATE OFFICES BY 11 NRC PERSONNEL. A SUMMATION OF THE LICENSEE PERFORMANCE EVALUATION WAS PRESENTED. AREAS OF CONCERN WERE DISCUSSED WITH CORPORATE MANAGEMENT. THE PERFORMANCE AT DRESDEN, QUAD-CITIES AND ZION NUCLEAR GENERATING STATIONS WAS CONSIDERED TO BE ADEQUATE.

REVIEWED BY OMPA LOEB: John J. Rigg **DATE:** 1-16-81

REVIEWED BY NRR DL: Paul W O'Connor **DATE:** 1-19-81

DRESDEN 1

AVERAGE DAILY POWER LEVEL (MWe)

OPERATING STATUS

1. Docket Number: 50-010
2. Utility Data Prepared By. S. A. CIESZKIEWICZ (815) 942-2920 X489
3. Licensed Thermal Power (MWt): 700
4. Nameplate Rating (Gross MWe): 246.5 x 0.85 = 209
5. Design Electrical Rating (Net MWe): 200
6. Maximum Dependable Capacity (Gross MWe): 0
7. Maximum Dependable Capacity (Net MWe): 0
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons None
9. Power Level To Which Restricted, If Any (Net MWe) None
10. Reasons For Restrictions, If Any

SHUTDOWN INDEFINITELY

This Month Yr ulative

11. Hours In Reporting Period
12. Number Of Hours Reactor Was Critical
13. Reactor Reserve Shutdown Hours
14. Hours Generator On-
15. Unit Reserve Shutd rs
16. Gross Thermal Energy Generated (MWH)
17. Gross Electrical Energy Generated (MWH)
18. Net Electrical Energy Generated (MWH)
19. Unit Service Factor
20. Unit Availability Factor
21. Unit Capacity Factor (Using MDC Net) 1/
22. Unit Capacity Factor (Using DER Net) 1/
23. Unit Forced Outage Rate
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): None
25. If Shut Down At End Of Report Period, Estimated Date of Startup: 06/86

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
SEE SUMMARY BELOW.									

SUMMARY

DRESDEN 1 IS SHUTDOWN UNTIL APPROXIMATELY JUNE, 1986 FOR THE FOLLOWING REASONS:

1) HPCI INSTALLATION FOR ECCS UPGRADING
2) EQUIPMENT UPGRADE TO IEEE 279
3) MAJOR INSERVICE INSPECTION
4) CHEMICAL CLEANING
5) REFUEL

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

DRESDEN 2

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Illinois/Grundy 9 miles E of Morris
2.	Electric Reliability Council	Mid-America Interpool Network
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2527
5.	Design Electrical Rating (MWE Net)	794
6.	Date of Initial Criticality	January 7, 1970
7.	Date of Commercial Operation	JULY, 1970
8.	Condenser Cooling Method	Cooling Lake
9.	Condenser Cooling Water Source	Kankakee and Des Plaines Rivers

Utility & Contractor Information

10.	Licensee	Commonwealth Edison Co.
11.	Corporate Address	P. O. Box 767 Chicago, Illinois 60690
12.	Corporate Contact	C. Reed
13.	Architect/Engineer	Sargent & Lundy
14.	Nuclear Steam Supply System	GE
15.	Constructor	United Engineers & Constructors, Inc.

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	R. Spessard
19.	IE Resident Inspector	T. Tongue (815) 942-9267
20.	Licensing Project Manager	Paul W. O'Connor
21.	Docket Number	50-237
22.	License Number/Date of Issue	DPR-19 issued 12/22/69
23.	Public Document Room	Morris Public Library 604 Liberty Street Morris, Illinois 60451

INSPECTION STATUS

ENFORCEMENT STATUS

10 CFR 50.54(K) STATES, "AN OPERATOR OR SENIOR OPERATOR LICENSED PURSUANT TO PART 55 OF THIS CHAPTER SHALL BE PRESENT AT THE CONTROLS AT ALL TIMES DURING THE OPERATION OF THE FACILITY." THE DRESDEN STATION TECHNICAL SPECIFICATIONS, CHAPTER 6.1 AND TABLE 6.1.1 REQUIRE, FOR THE OPERATIONAL STATUS OF UNITS 2 AND 3 ON AUGUST 8, 1980, THREE LICENSED OPERATORS TO FULFILL THE MINIMUM SHIFT STAFFING REQUIREMENTS. CONTRARY TO THE ABOVE, AT APPROXIMATELY 6:00 A.M. ON AUGUST 8, 1980, THE NRC SENIOR RESIDENT INSPECTOR OBSERVED THAT TWO OF THE THREE LICENSED OPERATORS IN THE CONTROL ROOM ASSIGNED TO MONITOR THE OPERATION OF UNITS 2 AND 3, SPECIFICALLY THE UNIT 3 AND CONTROL ROOM CENTER DESK OPERATORS, APPEARED TO BE ASLEEP, SUCH THAT THEY WERE NOT FULLY ATTENTIVE NOR WERE THEY ABLE TO MONITOR REACTOR CONDITIONS; THUS, LICENSED OPERATORS WERE NOT AT THE CONTROLS AS REQUIRED.
(8016 DEFICIENCY)

THE LICENSEE DID NOT CONDUCT PROPER BACKGROUND SCREENING OF CONTRACTOR PERSONNEL.
(8019 DEFICIENCY)

10 CFR 50, APPENDIX B, CRITERION II REQUIRES ACTIVITIES AFFECTING QUALITY BE ACCOMPLISHED UNDER SUITABLY CONTROLLED CONDITIONS, INCLUDING ADEQUATE CLEANNESS. THE LICENSEE'S QUALITY ASSURANCE PROGRAM, SECTION 2.2 REQUIRES THAT THE LICENSEE ADHERE TO ALL MANDATORY REQUIREMENTS OF ANSI N18.7. ANSI N18.7-1976, SECTION 5.2.10 REQUIRES QUALITY HOUSEKEEPING PRACTICES ENCOMPASSING ALL ACTIVITIES RELATED TO CONTROL OF FIRE PREVENTION AND PROTECTION, INCLUDING DISPOSAL OF COMBUSTIBLE MATERIAL AND DEBRIS. CONTRARY TO THE ABOVE, DURING ROUTINE PLANT TOURS ON SEPTEMBER 19, 26, AND OCTOBER 3, 1980, THE INSPECTORS IDENTIFIED SEVERAL AREAS OF ACCUMULATED COMBUSTIBLE MATERIAL AND DEBRIS WHICH WERE NOT BEING CONTROLLED AND WHICH REPRESENTED A POTENTIAL FIRE HAZARD. EXAMPLES INCLUDED: SEPTEMBER 19, 1980, UNIT 1 TURBINE BUILDING TRACK WAY AND TURBINE FLOOR - PRESENCE OF PAPER, WOOD, PLASTIC AND SLINGS ON THE FLOOR AND UNIT 2 REACTOR BUILDING, R.W.C.U. DEMINERALIZER RECIRCULATION PUMP ROOM - PRESENCE OF MOP HEADS, RAGS, PAPER ON THE FLOOR AND GENERALLY DIRTY; ON SEPTEMBER 26, 1980, UNIT 3 REACTOR BUILDING, GROUND FLOOR WEST - PRESENCE OF EXCESSIVE SWP SHOE COVERS, WIPES, SCRAPS OF PAPER, PLASTIC AND GENERALLY DIRTY; AND ON OCTOBER 3, 1980, UNIT 2 LPCI ROOMS - PRESENCE OF USED SWP CLOTHING, PLASTIC BAGS, WIPES, AND A BAG OF TRASH AND UNIT 2 TORUS BASEMENT - PRESENCE OF HOSES, WIRES, CABLES, WOOD, PAPER, PLASTIC, DEBRIS AND RUBBLE IN LARGE QUANTITIES SHOWING EVIDENCE OF HAVING BEEN THERE FOR A LONG TIME. THIS IS A REPETITIVE ITEM OF NONCOMPLIANCE SINCE A SIMILAR PROBLEM WAS IDENTIFIED ONCE PREVIOUSLY IN NRC INSPECTION REPORT NO. 50-10/80-09; 50-237/80-08; 50-249/80-12, DATED MAY 23, 1980.
(3021 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

UNIT OPERATING ROUTINELY IN COASTDOWN MODE.

+LAST IE SITE INSPECTION DATE: OCTOBER 31, 1980

+ INSPECTION REPORT NO: 80-22

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-38/ 03L-0	10/11/80	10/21/80	CRD BECAME UNCOUPLED BECAUSE OF LOOSE INNER FILTER
80-39/ 03X-0	10/11/80	10/29/80	HPCI STEAM VALVE FAILED TO OPEN AS A RESULT OF LUBRICANT RELATED TORQUE SWITCH FAILURE.
80-40/ 03L-0	10/12/80	10/22/80	CRD BECAME UNCOUPLED BECAUSE OF LOOSE INNER FILTER.

SUMMARY

INSPECTION SUMMARY

+ INVESTIGATION ON SEPTEMBER 8-10, (80-16): BECAUSE TWO REACTOR OPERATORS WERE CONSIDERED BY THE NRC SENIOR RESIDENT INSPECTOR TO BE ASLEEP WHILE ON DUTY IN THE REACTOR CONTROL ROOM, REVIEWED RECORDS, INTERVIEWED PERSONNEL AND OBTAINED SWORN STATEMENTS. THE INVESTIGATION INVOLVED 20 INVESTIGATION-HOURS ONSITE BY ONE NRC INVESTIGATOR. THE INVESTIGATION DETERMINED THAT AN NRC INSPECTOR OBSERVED TWO OPERATORS APPARENTLY ASLEEP, BUT BOTH OPERATORS, WHILE STATING THEY MAY HAVE CLOSED THEIR EYES OR "NODDED OFF", STATED THEY WERE NOT ASLEEP. BOTH OPERATORS CLAIMED TO BE READING AT THE TIME. ONE ITEM OF NONCOMPLIANCE WAS IDENTIFIED IN THAT CONTRARY TO 10 CFR 50.54(K), TWO LICENSED OPERATORS APPEARED TO BE ASLEEP SUCH THAT THEY WERE NOT FULLY ATTENTIVE NOR ABLE TO MONITOR REACTOR CONDITIONS.

+ INSPECTION ON SEPTEMBER 7 THROUGH OCTOBER 3, (80-21): ROUTINE, UNANNOUNCED RESIDENT INSPECTION OF FOLLOWUP ON PREVIOUS NONCOMPLIANCE; MONTHLY SURVEILLANCE OBSERVATION; MONTHLY MAINTENANCE OBSERVATION; INSPECTION DURING LONG-TERM SHUT DOWN; IE BULLETIN 80-06 FOLLOWUP; AND, FOLLOWUP ON HEADQUARTERS REQUEST. THE INSPECTION INVOLVED 83 INSPECTOR-HOURS ONSITE BY TWO NRC INSPECTORS INCLUDING 15 HOURS ONSITE DURING OFF-SHIFTS. OF THE SEVEN AREAS INSPECTED, THERE WERE NO ITEMS OF NONCOMPLIANCE IN SIX AREAS. THERE WAS AN APPARENT ITEM OF NONCOMPLIANCE (FAILURE TO ADHERE TO QA PROGRAM REQUIREMENTS) IN ONE AREA.

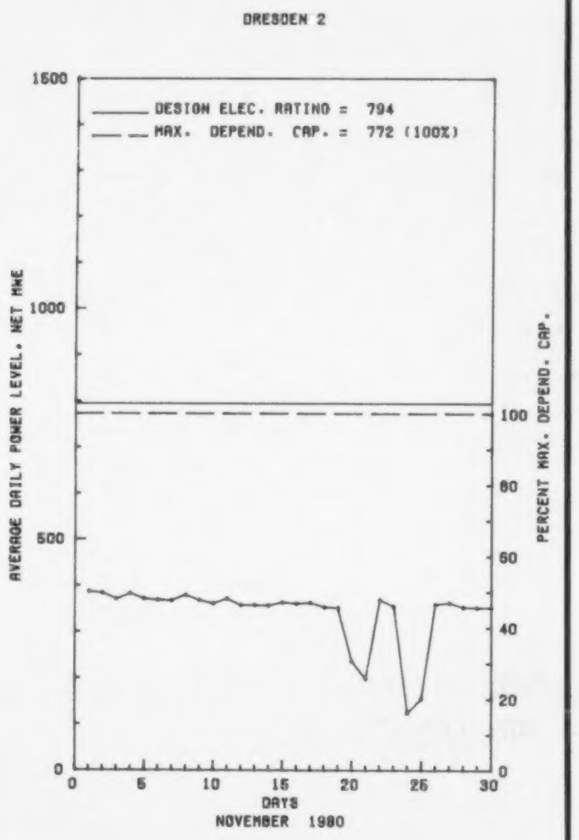
+ MANAGEMENT MEETING ON OCTOBER 31, (80-22): MANAGEMENT MEETING HELD AT THE NRC'S REQUEST TO DISCUSS THE REGULATORY PERFORMANCE OF THE ACTIVITIES AT DRESDEN, QUAD-CITIES, AND ZION NUCLEAR GENERATING STATIONS AS CONCLUDED IN THE SYSTEMATIC ASSESSMENT OF LICENSEE PERFORMANCE (SALP) PROGRAM. THE MEETING INVOLVED 18 MAN-HOURS AT THE CORPORATE OFFICES BY 11 NRC PERSONNEL. A SUMMATION OF THE LICENSEE PERFORMANCE EVALUATION WAS PRESENTED. AREAS OF CONCERN WERE DISCUSSED WITH CORPORATE MANAGEMENT. THE PERFORMANCE AT DRESDEN, QUAD-CITIES AND ZION NUCLEAR GENERATING STATIONS WAS CONSIDERED TO BE ADEQUATE.

REVIEWED BY OMPA LOEB: John J. [illegible] **DATE:** 1-16-81

REVIEWED BY NRR DL: Paul W O'Connor **DATE:** 1-19-81

DRESDEN 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-237
2. Utility Contact: S. A. CIESZKIEWICZ (815) 942-2920 X489
3. Licensed Thermal Power (MWt): 2527
4. Nameplate Rating (Gross MWe): 920 x 0.9 = 828
5. Design Electrical Rating (Net MWe): 794
6. Maximum Dependable Capacity (Gross MWe): 812
7. Maximum Dependable Capacity (Net MWe): 772
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	8,040.0	97,496.0
12. Hours Reactor Was Critical	691.4	7,741.3	72,478.0
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	674.7	7,571.3	68,834.0
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	868,020	14,933,906	136,575,738
17. Gross Electrical Energy (MWH)	263,069	4,649,397	43,656,018
18. Net Electrical Energy (MWH)	243,039	4,382,271	41,270,383
19. Unit Service Factor	93.7	94.2	74.4
20. Unit Availability Factor	93.7	94.2	74.4
21. Unit Capacity Factor (MDC Net)	43.7	70.6	57.8
22. Unit Capacity Factor (DER Net)	42.5	68.6	56.2
23. Unit Forced Outage Rate	6.3	3.3	12.4

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
REFUELING, DECEMBER 31, 1980, 17 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
10	11/20/80	F	17.5	A	2				MANUALLY SCRAMMED REACTOR - 2B CRD PUMP TRIPPED AND FAILED TO RESTART.
11	11/24/80	F	27.8	G	3				REACTOR SCRAMMED ON GROUP I ISOLATION WHILE I.M.'S WERE PERFORMING MSL HI FLOW SURVEILLANCE DIS 250-1.

SUMMARY

DRESDEN 2 OPERATED WITH 2 OUTAGES FOR OPERATIONAL ERROR AND EQUIPMENT FAILURE. POWER OUTPUT IS DECLINING DUE TO REFUELING WHICH SHOULD BEGIN DEC. 31, 1980.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for License Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

DRESDEN 3

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Illinois/Grundy 9 miles E of Morris
2.	Electric Reliability Council	Mid-America Interpool Network
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2527
5.	Design Electrical Rating (MWE Net)	794
6.	Date of Initial Criticality	January 31, 1971
7.	Date of Commercial Operation	November 16, 1971
8.	Condenser Cooling Method	Cooling Lake
9.	Condenser Cooling Water Source	Kankakee and Des Plaines Rivers

Utility & Contractor Information

10.	Licensee	Commonwealth Edison Co.
11.	Corporate Address	P. O. Box 767 Chicago, Illinois 60690
12.	Corporate Contact	Cordell Reed, Vice President
13.	Architect/Engineer	Sargent & Lundy
14.	Nuclear Steam Supply System	GE
15.	Constructor	United Engineers & Constructors, Inc.

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	R. Spessard
19.	IE Resident Inspector	T. Tongue 815-942-9267
20.	Licensing Project Manager	R. Bevan
21.	Docket Number	50-249
22.	License Number/Date of Issue	DPR-25 issued 1/12/71
23.	Public Document Room	Morris Public Library 604 Liberty Street Morris, Illinois 60451

INSPECTION STATUS

ENFORCEMENT STATUS

10 CFR 50.54(K) STATES, "AN OPERATOR OR SENIOR OPERATOR LICENSED PURSUANT TO PART 55 OF THIS CHAPTER SHALL BE PRESENT AT THE CONTROLS AT ALL TIMES DURING THE OPERATION OF THE FACILITY." THE DRESDEN STATION TECHNICAL SPECIFICATIONS, CHAPTER 6.1 AND TABLE 6.1.1 REQUIRE, FOR THE OPERATIONAL STATUS OF UNITS 2 AND 3 ON AUGUST 8, 1980, THREE LICENSED OPERATORS TO FULFILL THE MINIMUM SHIFT STAFFING REQUIREMENTS. CONTRARY TO THE ABOVE, AT APPROXIMATELY 6:00 A.M. ON AUGUST 8, 1980, THE NRC SENIOR RESIDENT INSPECTOR OBSERVED THAT TWO OF THE THREE LICENSED OPERATORS IN THE CONTROL ROOM ASSIGNED TO MONITOR THE OPERATION OF UNITS 2 AND 3, SPECIFICALLY THE UNIT 3 AND CONTROL ROOM CENTER DESK OPERATORS, APPEARED TO BE ASLEEP, SUCH THAT THEY WERE NOT FULLY ATTENTIVE NOR WERE THEY ABLE TO MONITOR REACTOR CONDITIONS; THUS, LICENSED OPERATORS WERE NOT AT THE CONTROLS AS REQUIRED.
(8020 DEFICIENCY)

THE LICENSEE DID NOT CONDUCT PROPER BACKGROUND SCREENING OF CONTRACTOR PERSONNEL.
(8023 DEFICIENCY)

10 CFR 50, APPENDIX B, CRITERION II REQUIRES ACTIVITIES AFFECTING QUALITY BE ACCOMPLISHED UNDER SUITABLY CONTROLLED CONDITIONS, INCLUDING ADEQUATE CLEANNESS. THE LICENSEE'S QUALITY ASSURANCE PROGRAM, SECTION 2.2 REQUIRES THAT THE LICENSEE ADHERE TO ALL MANDATORY REQUIREMENTS OF ANSI N18.7. ANSI N18.7-1976, SECTION 5.2.10 REQUIRES QUALITY HOUSEKEEPING PRACTICES ENCOMPASSING ALL ACTIVITIES RELATED TO CONTROL OF FIRE PREVENTION AND PROTECTION, INCLUDING DISPOSAL OF COMBUSTIBLE MATERIAL AND DEBRIS. CONTRARY TO THE ABOVE, DURING ROUTINE PLANT TOURS ON SEPTEMBER 19, 26, AND OCTOBER 3, 1980, THE INSPECTORS IDENTIFIED SEVERAL AREAS OF ACCUMULATED COMBUSTIBLE MATERIAL AND DEBRIS WHICH WERE NOT BEING CONTROLLED AND WHICH REPRESENTED A POTENTIAL FIRE HAZARD. EXAMPLES INCLUDED: SEPTEMBER 19, 1980, UNIT 1 TURBINE BUILDING TRACK WAY AND TURBINE FLOOR - PRESENCE OF PAPER, WOOD, PLASTIC AND SLINGS ON THE FLOOR AND UNIT 2 REACTOR BUILDING, R.W.C.U. DEMINERALIZER RECIRCULATION PUMP ROOM - PRESENCE OF MOP HEADS, RAGS, PAPER ON THE FLOOR AND GENERALLY DIRTY; ON SEPTEMBER 26, 1980, UNIT 3 REACTOR BUILDING, GROUND FLOOR WEST - PRESENCE OF EXCESSIVE SWP SHOE COVERS, WIPES, SCRAPS OF PAPER, PLASTIC AND GENERALLY DIRTY; AND ON OCTOBER 3, 1980, UNIT 2 LPCI ROOMS - PRESENCE OF USED SWP CLOTHING, PLASTIC BAGS, WIPES, AND A BAG OF TRASH AND UNIT 2 TORUS BASEMENT - PRESENCE OF HOSES, WIRES, CABLES, WOOD, PAPER, PLASTIC, DEBRIS AND RUBBLE IN LARGE QUANTITIES SHOWING EVIDENCE OF HAVING BEEN THERE FOR A LONG TIME. THIS IS A REPETITIVE ITEM OF NONCOMPLIANCE SINCE A SIMILAR PROBLEM WAS IDENTIFIED ONCE PREVIOUSLY IN NRC INSPECTION REPORT NO. 50-10/80-09; 50-237/80-08; 50-249/80-12, DATED MAY 23, 1980.
(8025 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

UNIT OPERATING ROUTINELY.

+LAST IE SITE INSPECTION DATE: OCTOBER 31, 1980

+ INSPECTION REPORT NO: 80-26

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-35/ 03L-0	10/09/80	10/29/80	FOLLOWING A UNIT 2 SCRAM AND A GROUP II ISOLATION, "B" SBGTS TRIPPED AFTER AN AUTO START.
80-36/ 03L-0	10/09/80	10/23/80	RBM COMPONENT FAILURE PREVENTING AUTO TRIP POINT ADJUSTMENT.
80-37/ 03L-0	10/20/80	11/17/80	DURING NORMAL OPERATION WITH DIS 500-2 IN PROGRESS, LEVEL SWITCH LIS 263-57B WAS FOUND TO TRIP AT A DIFFERENTIAL PRESSURE WHICH CORRESPONDED TO A LEVEL SET PIONT OF LESS THAN THE T.S. LIMIT OF 84 INCHES.
80-40/ 01T-0	10/11/80	10/28/80	HPCI FOUND INOPERABLE DURING TEST AS A RESULT OF DIRTY AUX. CONTACT PREVENTING STEAM SUPPLY VALVE FROM OPENING.

SUMMARY

INSPECTION SUMMARY

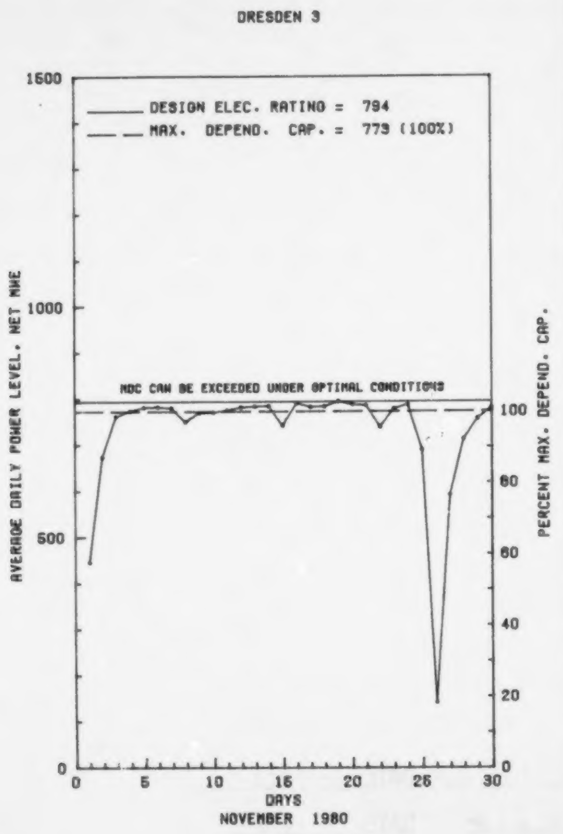
- INVESTIGATION ON SEPTEMBER 8-10, (80-20): BECAUSE TWO REACTOR OPERATORS WERE CONSIDERED BY THE NRC SENIOR RESIDENT INSPECTOR TO BE ASLEEP WHILE ON DUTY IN THE REACTOR CONTROL ROOM, REVIEWED RECORDS, INTERVIEWED PERSONNEL AND OBTAINED SWORN STATEMENTS. THE INVESTIGATION INVOLVED 20 INVESTIGATION-HOURS ONSITE BY ONE NRC INVESTIGATOR. THE INVESTIGATION DETERMINED THAT AN NRC INSPECTOR OBSERVED TWO OPERATORS APPARENTLY ASLEEP, BUT BOTH OPERATORS, WHILE STATING THEY MAY HAVE CLOSED THEIR EYES OR "NODDED OFF", STATED THEY WERE NOT ASLEEP. BOTH OPERATORS CLAIMED TO BE READING AT THE TIME. ONE ITEM OF NONCOMPLIANCE WAS IDENTIFIED IN THAT CONTRARY TO 10 CFR 50.54(K), TWO LICENSED OPERATORS APPEARED TO BE ASLEEP SUCH THAT THEY WERE NOT FULLY ATTENTIVE NOR ABLE TO MONITOR REACTOR CONDITIONS.
- INSPECTION ON SEPTEMBER 7 THROUGH OCTOBER 3, (80-25): ROUTINE, UNANNOUNCED RESIDENT INSPECTION OF FOLLOWUP ON PREVIOUS NONCOMPLIANCE; MONTHLY SURVEILLANCE OBSERVATION; MONTHLY MAINTENANCE OBSERVATION; INSPECTION DURING LONG-TERM SHUT DOWN; IE BULLETIN 80-06 FOLLOWUP; AND, FOLLOWUP ON HEADQUARTERS REQUEST. THE INSPECTION INVOLVED 83 INSPECTOR-HOURS ONSITE BY TWO NRC INSPECTORS INCLUDING 15 HOURS ONSITE DURING OFF-SHIFTS. OF THE SEVEN AREAS INSPECTED, THERE WERE NO ITEMS OF NONCOMPLIANCE IN SIX AREAS. THERE WAS AN APPARENT ITEM OF NONCOMPLIANCE (FAILURE TO ADHERE TO QA PROGRAM REQUIREMENTS) IN ONE AREA.
- MANAGEMENT MEETING ON OCTOBER 31, (80-26): MANAGEMENT MEETING HELD AT THE NRC'S REQUEST TO DISCUSS THE REGULATORY PERFORMANCE OF THE ACTIVITIES AT DRESDEN, QUAD-CITIES, AND ZION NUCLEAR GENERATING STATIONS AS CONCLUDED IN THE SYSTEMATIC ASSESSMENT OF LICENSEE PERFORMANCE (SALP) PROGRAM. THE MEETING INVOLVED 18 MAN-HOURS AT THE CORPORATE OFFICES BY 11 NRC PERSONNEL. A SUMMATION OF THE LICENSEE PERFORMANCE EVALUATION HAS PRESENTED. AREAS OF CONCERN WERE DISCUSSED WITH CORPORATE MANAGEMENT. THE PERFORMANCE AT DRESDEN, QUAD-CITIES AND ZION NUCLEAR GENERATING STATIONS WAS CONSIDERED TO BE ADEQUATE.

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 1-16-81

REVIEWED BY NRR DL: R.B.Bevan **DATE:** 1/19/81

DRESDEN 3

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-249
2. Utility Contact: S. A. CIESZKIEWICZ (815) 942-2920 X489
3. Licensed Thermal Power (MWt): 2527
4. Nameplate Rating (Gross MWe): 920 x 0.9 = 828
5. Design Electrical Rating (Net MWe): 794
6. Maximum Dependable Capacity (Gross MWe): 812
7. Maximum Dependable Capacity (Net MWe): 773
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	8,040.0	82,081.0
12. Hours Reactor Was Critical	710.4	5,793.1	61,558.5
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	706.2	5,691.8	59,016.4
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,677,311	12,837,692	115,109,360
17. Gross Electrical Energy (MWH)	549,017	4,120,912	37,550,586
18. Net Electrical Energy (MWH)	522,414	3,903,453	35,596,744
19. Unit Service Factor	98.1	70.8	71.9
20. Unit Availability Factor	98.1	70.8	71.9
21. Unit Capacity Factor (MDC Net)	93.9	62.8	56.1
22. Unit Capacity Factor (DER Net)	91.4	61.1	54.6
23. Unit Forced Outage Rate	1.9	2.5	15.3

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
TMI MODIFICATIONS, DEC. 15, 1980, 4 OR 5 DAYS TENIATIVE.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
15	11/26/80	F	13.8	A	3				REACTOR SCRAMMED ON GROUP I ISOLATION. REACTOR MODE SWITCH CONTACTS DID NOT MAKE UP PROPERLY WHEN PLACED FROM RUN TO STARTUP AND THE UNIT WENT BELOW 850# PRESSURE.

SUMMARY

DRESDEN 3 OPERATED NORMALLY WITH 1 OUTAGE ON THE 26TH DUE TO EQUIPMENT FAILURE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

DUANE ARNOLD

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Iowa/Linn 8 miles NW of Cedar Rapids
2.	Electric Reliability Council	Mid-Continent Area Reliability Coordination Agreement
3.	Type of Reactor	BWR
4.	Power Level (MWT)	1658
5.	Design Electrical Rating (MWE Net)	538
6.	Date of Initial Criticality	March 23, 1974
7.	Date of Commercial Operation	February 1, 1975
8.	Condenser Cooling Method	Cooling Towers
9.	Condenser Cooling Water Source	Cedar Rapids River Circulating Water System

Utility & Contractor Information

10.	Licensee	Iowa Electric Light and Power
11.	Corporate Address	IE Towers, P. O. Box 351 Cedar Rapids, Iowa 52406
12.	Corporate Contact	D. Arnold, President
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	GE
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	W. S. LITTLE
19.	IE Resident Inspector	W. F. CHRISTIANSON
20.	Licensing Project Manager	K. T. Eccleston
21.	Docket Number	50-331
22.	License Number/Date of Issue	DPR-49 issued 2/22/74
23.	Public Document Room	Reference Service Cedar Rapids Public Library 428 Third Avenue, S.E. Cedar Rapids, Iowa 52401

INSPECTION STATUS

ENFORCEMENT STATUS

ENVIRONMENTAL TECHNICAL SPECIFICATIONS 5.4.LIII AND 5.4.LIV REQUIRE, IN PART, REPORTING OF TOTAL GROSS RADIOACTIVITY AND PERCENT OF TECHNICAL SPECIFICATION LIMITS FOR NUCLIDES"...WITH HALF-LIVES GREATER THAN EIGHT DAYS." CONTRARY TO THE ABOVE, BA-140, WHICH WAS NOT IN THE LICENSEE'S NUCLIDE LIBRARY, WAS NOT REPORTED FOR ALL THE INSTANCES WHERE IT WAS A CONSTITUENT OF A PARTICULATE RELEASE.
(8016 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENTS

+ IE BULLETIN 80-07, BWR JET PUMP BEAM ASSEMBLY. UT EXAMINATION COMPLETED ON APRIL 4, 1980 BY GE EXAMINATION TEAM, BULLETIN CLOSED. IE BULLETIN 80-08, CONTAINMENT PENETRATIONS. BULLETIN REQUIREMENT COMPLETE AND REVIEWED BY NRC INSPECTOR, BULLETIN CLOSED. (80-19)

MANAGERIAL ITEMS

+ MR. R. DYE ACCEPTED THE POSITION OF PLANT CHEMIST EFFECTIVE IN JULY 1980. (80-16)

PLANT STATUS

UNIT OPERATING ROUTINELY.

+LAST IE SITE INSPECTION DATE: OCTOBER 14-16, 1980

+ INSPECTION REPORT NO: 80-20

SUMMARY

INSPECTION SUMMARY

+ INSPECTION ON SEPTEMBER 1-30, (80-15): ROUTINE INSPECTION, REVIEW AND OBSERVATIONS OF PLANT OPERATIONS AND MAINTENANCE, PROCEDURES, DOCUMENTATION AND WORK PERFORMED. THE INSPECTION INVOLVED 237 INSPECTOR-HOURS ONSITE BY TWO NRC INSPECTORS INCLUDING 22 INSPECTOR-HOURS ONSITE DURING OFF-SHIFTS. NO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED.

+ INSPECTION ON SEPTEMBER 8-11 AND OCTOBER 15-16, (80-16): ROUTINE, UNANNOUNCED INSPECTION OF: (1) THE ENVIRONMENTAL PROTECTION PROGRAM, INCLUDING MANAGEMENT CONTROLS, PROGRAM REVIEW AND AUDIT, RADIOLOGICAL AND NONRADIOLOGICAL MONITORING PROGRAM, WATER QUALITY MONITORING AND SPECIAL STUDIES, (2) CONFIRMATORY MEASUREMENTS PROGRAM, INCLUDING DISCUSSION OF RESULTS OF PREVIOUS RADIOLOGICAL EFFLUENT SAMPLES AND COLLECTION AND ANALYSIS OF RADIOLOGICAL EFFLUENT SAMPLES IN THE NRC MEASUREMENTS VAN ONSITE, AND (3) REVIEW OF ENVIRONMENTAL TECHNICAL SPECIFICATION VIOLATIONS. THE INSPECTION INVOLVED 45 INSPECTOR-HOURS ONSITE BY TWO NRC INSPECTORS. OF THE EIGHT AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN SEVEN AREAS; ONE APPARENT ITEM OF NONCOMPLIANCE WAS IDENTIFIED IN ONE AREA (FAILURE TO REPORT REQUIRED NUCLIDES IN EFFLUENT).

+ INSPECTION ON SEPTEMBER 29 THROUGH OCTOBER 2 AND 21-23, (80-19): TORUS SUPPORT REINFORCEMENT INSTALLATION, WELDING, AND NONDESTRUCTIVE EXAMINATIONS; IE BULLETIN 80-07 AND IE BULLETIN 80-08 CORRECTIVE ACTIONS. THE INSPECTION INVOLVED 44 INSPECTOR-HOURS ONSITE BY ONE NRC INSPECTOR. NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

+ INSPECTION ON OCTOBER 14-16, (80-20): ANNOUNCED INSPECTION OF THE LICENSEE'S IMPLEMENTATION OF THE QUALITY ASSURANCE PROGRAM IN THE FOLLOWING AREAS: RECEIPT, STORAGE AND HANDLING; PROCUREMENT CONTROL; AND FOLLOWUP ON PREVIOUS INSPECTION FINDINGS. THE INSPECTION INVOLVED 38 INSPECTOR-HOURS ONSITE BY TWO NRC INSPECTORS. NO INSPECTOR-HOURS WERE ONSITE DURING OFF-SHIFTS. OF THE AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

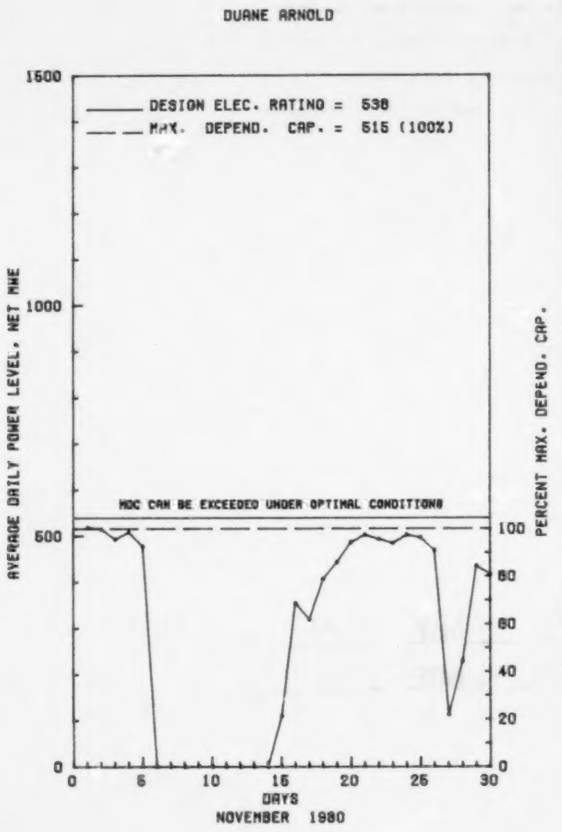
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-50/ 03L-0	09/17/80	10/16/80	RCIC STEAM SUPPLY LINE VALVE WOULD NOT OPEN.

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 1-16-81

REVIEWED BY NRR DL: **DATE:** 1-19-81

DUANE ARNOLD

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-331
2. Utility Contact: J. Van Sickel (319) 851-5611
3. Licensed Thermal Power (MWt): 1658
4. Nameplate Rating (Gross MWe): 663 x 0.9 = 597
5. Design Electrical Rating (Net MWe): 538
6. Maximum Dependable Capacity (Gross MWe): 545
7. Maximum Dependable Capacity (Net MWe): 515
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	8,040.9	51,120.0
12. Hours Reactor Was Critical	512.9	5,877.3	36,441.4
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	482.4	5,734.1	35,553.2
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	660,358	7,760,578	44,820,138
17. Gross Electrical Energy (MWH)	224,109	2,600,088	15,024,446
18. Net Electrical Energy (MWH)	210,988	2,439,719	14,054,778
19. Unit Service Factor	67.0	71.3	69.5
20. Unit Availability Factor	67.0	71.3	69.5
21. Unit Capacity Factor (MDC Net)	56.9	58.9	53.4
22. Unit Capacity Factor (DER Net)	54.5	56.4	51.1
23. Unit Forced Outage Rate	33.0	9.4	19.3

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
REFUELING, MARCH 7, 1980, 9 WEEKS
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
20	11/06/80	F	220.8	A	1		CB	PUMPXX	PLANT WAS SHUTDOWN TO REPLACE THE SEAL ON THE "A" RECIRCULATION PUMP. DURING STARTUP ON 11/10/80 MAIN STEAM RELIEF VALVE PSV 4405 WAS OPENED AND STUCK OPEN RESULTING IN A REACTOR SHUTDOWN. PLANT REMAINED SHUTDOWN PENDING REPAIRS TO PSV-4405 AND CORRECTION OF A WIRING ERROR.
21	11/27/80	F	8.6	A	4		HA	PIPEXX	POWER WAS REDUCED AND THE GENERATOR TAKEN OFF LINE TO REPAIR AN EHC OIL LEAK.
22	11/27/80	F	8.2	G	3				REACTOR SCRAM WHEN OPERATOR SET IN WRONG APRM GAIN ADJUSTMENT.

SUMMARY

THE PLANT OPERATED ROUTINELY EXCEPT FOR THE OUTAGES AND POWER REDUCTIONS LISTED IN DETAIL ABOVE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

FARLEY 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Alabama/Houston 24 miles SE of Dothan
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2652
5.	Design Electrical Rating (MWE Net)	829
6.	Date of Initial Criticality	August 9, 1977
7.	Date of Commercial Operation	December 1, 1977
8.	Condenser Cooling Method	Once Through With Forced Draft Cooling Towers
9.	Condenser Cooling Water Source	Chattahoochee River

Utility & Contractor Information

10.	Licensee	Alabama Power Company
11.	Corporate Address	800 North 18th Street Birmingham, Alabama
12.	Corporate Contact	F. L. Clayton, Jr. Senior Vice President (205-323-5341)
13.	Architect/Engineer	Bechtel/SSI
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Daniel

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. Lewis
18.	IE Section Chief	R. D. Martin
19.	IE Resident Inspector	W. H. Bradford (205-899-3386)
20.	Licensing Project Manager	E. A. Reeves
21.	Docket Number	50-348
22.	License Number/Date of Issue	NPF-2, 6/25/77
23.	Public Document Room	G. S. Houston Memorial Library 212 W. Berdeshaw Street Dothan, Alabama 36301

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

+ REFUELING OUTAGE SCHEDULED TO END MID-JANUARY.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

+ SHUT DOWN FOR REFUELING.
+ LAST IE SITE INSPECTION DATE: NOVEMBER 1-30, 1980
+ INSPECTION REPORT NO: 50-348/80-40

SUMMARY

INSPECTION SUMMARY

+ INSPECTION SEPTEMBER 16-19 (80-28): THIS SPECIAL, ANNOUNCED PERFORMANCE APPRAISAL INSPECTION OF THE LICENSEE'S PROGRAM AND DOCUMENTATION ASSOCIATED WITH THE CONTROL OF CENTRIFUGAL CHARGING/SAFETY INJECTION PUMPS MANUFACTURED BY PACIFIC PUMP DIVISION OF DRESSER INDUSTRIES. THIS INSPECTION INVOLVED 25 INSPECTOR-HOURS ON SITE BY ONE PERFORMANCE APPRAISAL BRANCH INSPECTOR. OF THE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION OCTOBER 7 (80-30): A CORPORATE MANAGEMENT MEETING WAS HELD ON OCTOBER 7, 1980, TO DISCUSS THE ANTICIPATED RESOLUTIONS OF PROBLEMS IDENTIFIED IN THE UNIT 1 CONTAINMENT BUILDING TENDON SURVEILLANCE INSPECTIONS PERFORMED IN SPRING, 1978 AND SPRING, 1980. THIS MEETING INVOLVED 15 INSPECTOR MAN-HOURS.

INSPECTION OCTOBER 14-17 (80-31): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 13 INSPECTOR-HOURS ON SITE IN THE AREAS OF FIRE PROTECTION/PREVENTION. OF THE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE WERE FOUND.

REPORTS RECEIVED FROM LICENSEE

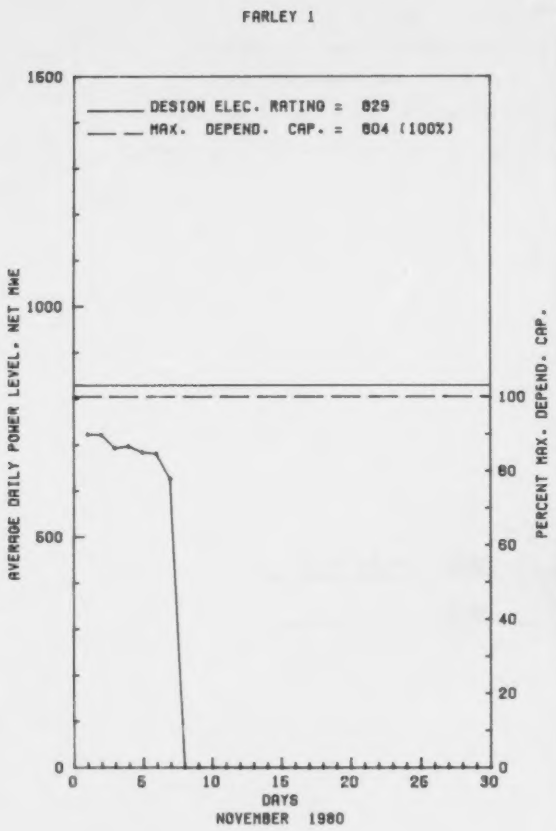
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-057/ 03L-0	09/22/80	10/22/80	THE RESIDUAL HEAT REMOVAL SYSTEM WAS MADE INOPERABLE DUE TO DEFECTIVE PROCEDURES
80-058/ 03L-0	10/07/80	11/04/80	UNEXPECTED TENDON LIFT-OFF FORCE REVEALED DURING CONTAINMENT TENDON SURVEILLANCE
80-059/ 03L-0	10/04/80	11/04/80	THE B TRAIN PENETRATION ROOM EXHAUST FAN FAILED TO START DUE TO A SHORTED RELAY
80-061/ 01T-0	11/04/80	11/17/80	FIRE PROTECTION WATER TANKS A AND B LEVELS WERE LOW
80-062/ 03L-0	10/17/80	11/17/80	DIESEL GENERATOR 1-2A WAS FOUND INOPERABLE DURING SURVEILLANCE TESTING

REVIEWED BY OMPA LOEB: PEBobe **DATE:** 1/16/81

REVIEWED BY NRR DL: **DATE:** [illegible]

FARLEY 1

AVERAGE POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-348
2. Utility Contact: W. G. Hairston, III (205) 899-5156
3. Licensed Thermal Power (MWt): 2652
4. Nameplate Rating (Gross MWe): 1045 x 0.85 = 888
5. Design Electrical Rating (Net MWe): 879
6. Maximum Dependable Capacity (Gross MWe): 845
7. Maximum Dependable Capacity (Net MWe): 804
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	8,040.0	26,304.0
12. Hours Reactor Was Critical	169.0	6,246.8	17,182.3
13. Reactor Reserve Shutdown Hours	.0	1,041.5	2,446.8
14. Hours Generator On-Line	168.0	6,112.9	16,703.4
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	382,570	15,391,876	41,939,891
17. Gross Electrical Energy (MWH)	122,664	4,895,370	13,419,498
18. Net Electrical Energy (MWH)	110,970	4,608,144	12,612,152
19. Unit Service Factor	23.3	76.0	63.5
20. Unit Availability Factor	23.3	76.0	63.5
21. Unit Capacity Factor (MDC Net)	19.2	71.3	59.6
22. Unit Capacity Factor (DER Net)	18.6	69.1	57.8
23. Unit Forced Outage Rate	.0	6.1	7.8

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 01/01/81

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
072	11/07/80	S	552.0	C	1		RC	ZZZZZZ	UNIT WAS TAKEN OFF LINE FOR REFUELING.

SUMMARY

FARLEY 1 OPERATED THRU NOVEMBER 7TH, AND THEN ENTERED INTO REFUELING OUTAGE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

NRC POWER RESTRICTIONS: NONE

FITZPATRICK

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	New York/Oswego 8 miles NE of Oswego
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2436
5.	Design Electrical Rating (MWE Net)	821
6.	Date of Initial Criticality	November 17, 1974
7.	Date of Commercial Operation	July 28, 1975
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Lake Ontario

Utility & Contractor Information

10.	Licensee	Power Authority of the State of New York
11.	Corporate Address	10 Columbus Circle New York, New York 10019
12.	Corporate Contact	J. P. Bayne, Manager of Nuclear Operations
13.	Architect/Engineer	Stone & Webster
14.	Nuclear Steam Supply System	GE
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Section Chief	H. B. KISTER
19.	IE Resident Inspector	W. H. BAUNACK (315) 342-4907 FTS: 264-3311
20.	Licensing Project Manager	P. Polk (X7872)
21.	Docket Number	50-333
22.	License Number/Date of Issue	DPR-59, 10/17/74
23.	Public Document Room	State University College of Oswego Penfield Library - Documents Oswego, NY 13126 (315)341-2323

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

+ SYSTEM AND COMPONENT PROBLEMS:

+ SCRAM DISCHARGE VALVE LEVEL MEASURING SYSTEM WAS INSTALLED AND DECLARED OPERATIONAL AS OF DECEMBER 1, 1980. HOWEVER, THE LICENSEE IS EXPERIENCING SPURIOUS ALARMS DUE TO CHATTER BETWEEN REDUNDANT CHANNELS.

PLANT STATUS:

+ ROUTINE POWER OPERATION WITH THE RECOMBINER OUT OF SERVICE. REPRESENTATIVE RELEASE RATES ARE 13,920 AND 5175 MICROCURIES PER SECOND RESPECTIVELY.

+ LAST IE SITE INSPECTION DATE: 10/25 - 11/30/80

+ INSPECTION REPORT NO: 50-333/80-21

SUMMARY

INSPECTION SUMMARY

+ NONE

REPORTS RECEIVED FROM LICENSEE

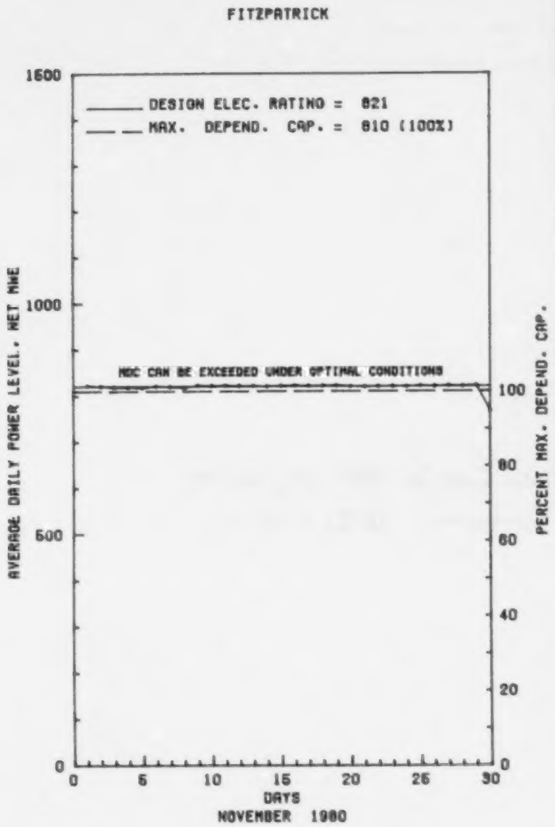
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-077/03L	10/03/80	11/03/80	DRYWELL EQUIPMENT DRAIN SUMP PUMP FAILED
80-078/03L	10/05/80	11/03/80	EQUIPMENT DRAIN SUMP INTEGRATOR AND FLOW RECORDER FOUND INOPERABLE WHILE PERFORMING SURVEILLANCE TEST
80-079/03L	10/08/80	11/07/80	RCIC SYSTEM MADE INOPERABLE TO REPAIR STEAM LEAKS
80-080/03L	10/11/80	11/07/80	PRIMARY CONTAINMENT ISOLATION VALVE FROM RHR TO RADWASTE FAILED TO CLOSE DURING SURVEILLANCE TEST
80-081/03L	10/23/80	11/21/80	FAILURE OF CARBON DIOXIDE SYSTEM IN NORTH CABLE ROOM TO ACTUATE
80-083/01T	11/18/80	11/19/80	OPERATION WITH ONLY ONE OPERABLE APRM IN RPS CHANNEL B DUE TO INSUFFICIENT LEVEL INPUTS TO AN APRM

REVIEWED BY OMPA LOEB: John J. [illegible] **DATE:** 1-16-81

REVIEWED BY NRR DL: Philip J. Polk **DATE:** 1/19/81

FITZPATRICK

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-333
2. Utility Contact: E. Zufelt (315) 342-3840
3. Licensed Thermal Power (MWt): 2436
4. Nameplate Rating (Gross MWe): 981 x 0.9 = 883
5. Design Electrical Rating (Net MWe): 821
6. Maximum Dependable Capacity (Gross MWe): 830
7. Maximum Dependable Capacity (Net MWe): 810
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

ITEM 21 CALCULATED WITH A WEIGHTED MDC

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	8,040.0	46,873.0
12. Hours Reactor Was Critical	720.0	5,539.8	32,170.0
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	720.0	5,420.8	31,158.8
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,743,216	11,239,608	62,427,802
17. Gross Electrical Energy (MWH)	608,850	3,861,290	21,214,990
18. Net Electrical Energy (MWH)	590,000	3,739,280	20,529,855
19. Unit Service Factor	100.0	67.4	66.5
20. Unit Availability Factor	100.0	67.4	66.5
21. Unit Capacity Factor (MDC Net)	101.2	58.1	58.4
22. Unit Capacity Factor (DER Net)	99.8	56.6	53.3
23. Unit Forced Outage Rate	.0	4.5	18.6

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): SNUBBER INSPECTION & MAINTENANCE OUTAGE -01/81 - 10DAYS
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
		NONE							

SUMMARY

UNIT OPERATED AT 100% RATED THERMAL POWER FOR THIS SUMMARY REPORTING PERIOD WITH NO OUTAGES OR REDUCTIONS.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

FORT CALHOUN

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1. Location (state/county) (nearest population center)	Nebraska/Washington 19 miles N of Omaha
2. Electric Reliability Council	Mid-Continent Area Reliability Coordination Agreement
3. Type of Reactor	PWR
4. Power Level (MWT)	1500
5. Design Electrical Rating (MWE Net)	490
6. Date of Initial Criticality	August 6, 1973
7. Date of Commercial Operation	June 20, 1974
8. Condenser Cooling Method	Once Through
9. Condenser Cooling Water Source	Missouri River

Utility & Contractor Information

10. Licensee	W. C. Jones Division Manager – Production Operations Omaha Public Power District
11. Corporate Address	1623 Harney Street Omaha, Nebraska 68102
12. Corporate Contact	T. E. Short Assistant General Manager
13. Architect/Engineer	Gibbs & Hill, Inc.
14. Nuclear Steam Supply System	Combustion Engineering
15. Constructor	Gibbs & Hill, Durham & Richardson, Inc.

Regulatory Information

16. IE Region responsible	IV
17. IE Branch Chief	G. L. Madsen
18. IE Section Chief	D. M. Hunnicutt
19. IE Resident Inspector	D. Kelley
20. Licensing Project Manager	P. C. Wagner
21. Docket Number	50-285
22. License Number/Date of Issue	DPR-40, 5/24/73
23. Public Document Room	W. Dale Clark Library 215 S. 15th Street Omaha, Nebraska 68102

INSPECTION STATUS

ENFORCEMENT STATUS

```
INSPECTION STATUS

OTHER ITEMS OF INTEREST
     SYSTEMS AND COMPONENT PROBLEMS:


     FACILITY ITEMS (PLANS AND PROCEDURES):



     MANAGERIAL ITEMS:



     PLANT STATUS:
     NO SIGNIFICANT INFORMATION

LAST IE SITE INSPECTION DATE:  OCTOBER 1 - 31, 1980

INSPECTION REPORT NO:  50-285/80-20
```

```
SUMMARY

INSPECTION SUMMARY

INSPECTION ON SEPTEMBER 30 - OCTOBER 3, 1980 (80-18):  ROUTINE,
UNANNOUNCED INSPECTION OF THE DOCUMENT CONTROL PROGRAM AND SURVEILLANCE
OF PIPE SUPPORT AND RESTRAINT SYSTEMS.  WITHIN THE TWO AREAS INSPECTED
NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE NOTED.

INSPECTION ON OCTOBER 21 - 24, 1980 (80-19):  ROUTINE, UNANNOUNCED
INSPECTION OF THE SURVEILLANCE TESTING AND CALIBRATION CONTROL PRO-
GRAM.  WITHIN THE ONE AREA INSPECTED, NO ITEMS OF NONCOMPLIANCE OR
DEVIATIONS WERE NOTED.

INSPECTION ON OCTOBER 1 - 31, 1980 (80-20):  ROUTINE, ANNOUNCED
INSPECTION OF OPERATIONAL SAFETY VERIFICATION, RADIOACTIVE WASTE
SHIPMENT, AND FOLLOWUP ON IE HEADQUARTERS REQUESTS.  WITHIN THE
THREE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS
WERE NOTED.
```

REPORTS RECEIVED FROM LICENSEE

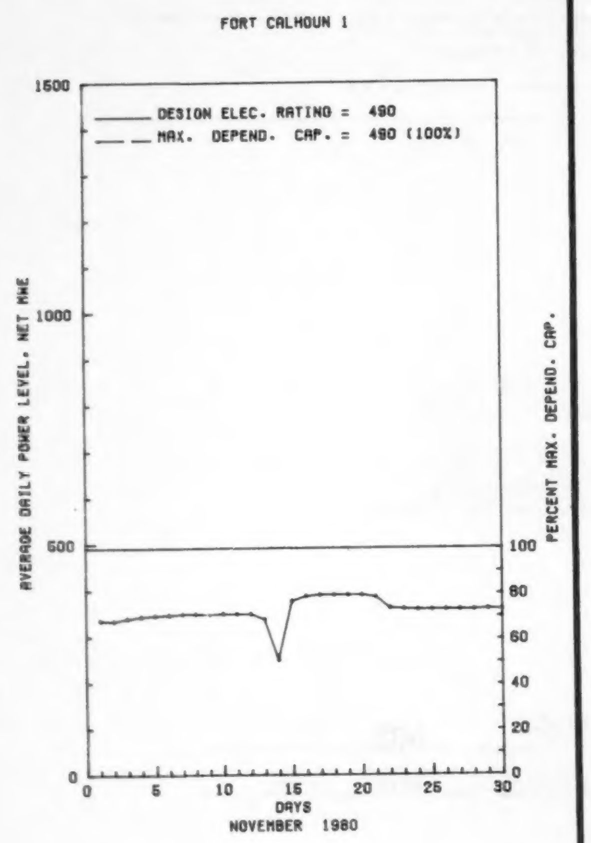
```
==========================================================================================
NUMBER    DATE OF    DATE OF                          SUBJECT
           EVENT      REPORT
==========================================================================================
80-25     10/27/80   11/14/80     CHARGING PUMP DISCHARGE HEADER WELD JOINT LEAK
03L-0
80-26     10/30/80   10/30/80     RPS 8B-1 MATRIX RELAY FAILURE
03L-0
80-27     10/27/80   11/3/80      ESF-AC, SEQUENCER SI-2 FAILURE
03L-0
80-28     11/5/80    11/12/80     EMERGENCY DIESEL GENERATOR DG-1 ENGINE START FAILURE
03L-0
```

REVIEWED BY OMPA LOEB: M R Beebe **DATE:** 01/16/81

REVIEWED BY NRR DL: P C Wagner **DATE:** 01/16/81

FORT CALHOUN

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-285
2. Utility Contact: T. L. PATTERSON (402) 536-4413
3. Licensed Thermal Power (MWt): 1500
4. Nameplate Rating (Gross MWe): 591 X 0.85 = 502
5. Design Electrical Rating (Net MWe): 490
6. Maximum Dependable Capacity (Gross MWe): 513
7. Maximum Dependable Capacity (Net MWe): 490
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

ITEMS 21 & 22 CALCULATED WITH WEIGHTED VALUES

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	8,040.0	62,977.0
12. Hours Reactor Was Critical	720.0	4,632.7	49,034.3
13. Reactor Reserve Shutdown Hours	.0	.0	1,309.5
14. Hours Generator On-Line	716.1	4,565.9	48,016.8
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	889,219	5,665,775	58,284,802
17. Gross Electrical Energy (MWH)	271,130	1,846,580	19,506,810
18. Net Electrical Energy (MWH)	255,533	1,745,434	18,432,951
19. Unit Service Factor	99.5	56.8	76.2
20. Unit Availability Factor	99.5	56.8	76.2
21. Unit Capacity Factor (MDC Net)	72.4	46.9	64.3
22. Unit Capacity Factor (DER Net)	72.4	46.9	63.8
23. Unit Forced Outage Rate	.5	.4	4.1

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-04	11/14/80	F	3.9	A	4		CB	VALVEX	GENERATOR TAKEN OFF-LINE TO REPAIR PACKING LEAK ON REACTOR COOLANT SAMPLE LINE ROOT VALVE RC-138.

SUMMARY

FORT CALHOUN OPERATED AT ESSENTIALLY 75% POWER DURING THE MONTH OF NOVEMBER TO CONSERVE FUEL.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B- Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

FORT ST. VRAIN

FACILITY DATA

Facility Description

1. Location (state/county) (nearest population center)	Colorado/Weld 35 miles N of Denver (near Platteville)
2. Electric Reliability Council	Western Systems Coordinating Council
3. Type of Reactor	HTGR
4. Power Level (MWT)	842
5. Design Electrical Rating (MWE Net)	330
6. Date of Initial Criticality	January 31, 1974
7. Date of Commercial Operation	July 1, 1979
8. Condenser Cooling Method	Towers
9. Condenser Cooling Water Source	South Platte River/Shallow Wells

Utility & Contractor Information

10. Licensee	Public Service Company of Colorado
11. Corporate Address	P. O. Box 840 Denver, Colorado 80201
12. Corporate Contact	C. K. Millen Senior Vice President
13. Architect/Engineer	Sargent & Lundy
14. Nuclear Steam Supply System	General Atomic Company
15. Constructor	General Atomic Company

Regulatory Information

16. IE Region responsible	IV
17. IE Branch Chief	G. L. Madsen
18. IE Section Chief	T. F. Westerman
19. IE Resident Inspector	M. W. Dickerson (303) 785-2281
20. Licensing Project Manager	G. Kuzmycz
21. Docket Number	50-267
22. License Number/Date of Issue	DPR-34, 12/21/73
23. Public Document Room	Greeley Public Library City Complex Building Greeley, Colorado 80631

INSPECTION STATUS

ENFORCEMENT STATUS

10 CFR 50 APP B CRITERION II STATES IN PART, THE APPLICANT SHALL ESTABLISH AT THE EARLIST PRACTICABLE TIME A Q A PROGRAM. THIS PROGRAM SHALL BE DOCUMENTED BY WRITTEN POLICIES, PROCEDURES, OR INSTRUCTIONS AND SHALL BE CARRIED OUT THROUGHOUT PLANT LIFE IN ACCORDANCE WITH THOSE POLICIES, PROCEDURES OR INSTRUCTIONS. AMEND 32 TO THE FSAR (PUBLIC SERVICE COMPANY OF COLORADO APPROVED QA PROGRAM) SETS FORTH CERTAIN IMPLEMENTATION DATES FOR REVISING THE OVERALL PROGRAM. THE LICENSEE IN A LETTER TO THE NRC DATED DEC 17, 1976, COMMITTED TO ANSI N18.7-1972 TO BE EFFECTIVE DEC 2, 1975. SECT 5.1.5 OF ANSI N18.7-1972, "EQUIPMENT CONTROL PROCEDURES," REQUIRES THAT "PROCEDURES SHALL REQUIRE INDEPENDENT VERIFICATION, WHERE APPROPRIATE, TO ENSURE THAT NECESSARY MEASURES, SUCH AS TAGGING EQUIPMENT, HAVE BEEN IMPLEMENTED CORRECTLY." CONTRARY TO THE ABOVE THE LICENSEE'S EQUIPMENT CONTROL PROCEDURES DID NOT INCLUDE A REQUIREMENT THAT AN INDEPENDENT VERIFICATION BE PERFORMED TO ENSURE THAT NECESSARY MEASURES SUCH AS TAGGING EQUIPMENT, HAD BEEN IMPLEMENTED CORRECTLY. THIS IS A SEVERITY LEVEL VI VIOLATION. (SUPPLEMENT I.F) PURSUANT TO THE PROVISIONS OF 10 CFR 2.201, PUBLIC SERVICE CO OF COLORADO IS HEREBY REQUIRED TO SUBMIT TO THIS OFFICE WITHIN TWENTY-FIVE DAYS OF THE DATE OF THIS NOTICE, A WRITTEN STATEMENT OR EXPLANATION IN REPLY, INCLUDING (1) THE CORRECTIVE STEPS WHICH HAVE BEEN TAKEN AND THE RESULTS ACHIEVED; (2) CORRECTIVE STEPS WHICH WILL BE TAKEN TO AVOID FURTHER VIOLATIONS; AND (3) THE DATE WHEN FULL COMPLIANCE WILL BE ACHIEVED. UNDER THE AUTHORITY OF SECT 182 OF THE ATOMIC ENERGY ACT OF 1954, AS AMENDED, THIS RESPONSE SHALL BE SUBMITTED UNDER OATH OR AFFIRMATION.
(8020)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

FACILITY ITEMS (PLANS AND PROCEDURES):

MANAGERIAL ITEMS:

PLANT STATUS:

NO SIGNIFICANT PLANT CHANGES.

LAST IE SITE INSPECTION DATE: OCTOBER 31, 1980

INSPECTION REPORT NO: 50-267/80-21

NRC POWER RESTRICTIONS: Restricted to 70% of Licensed Thermal Power.

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-59 03L-0	10/12/80	11/10/80	OPERATED WITH TOTAL COOLANT OXIDANT GREATER THAN 10 PARTS PER MILLION AND CORE OUTLET GREATER THAN 1200 DEG F. LCO 4.2.10
80-60 03L-0	10/11/80	11/10/80	NONINTERRUPTIBLE BUS 3 INVERTER TRIPPED WHILE REACTOR WAS AT POWER. LCO 4.6.1
80-61 03L-0	10/3/80	10/31/80	PLANT OPERATED WITH INDIVIDUAL SNUBBERS OUT OF SERVICE FOR CORRECTIVE/ PREVENTIVE MAINTENANCE. LCO 4.3.10
80-62 01L-0	10/22/80	11/5/80	ISOLATION VALVE INOPERABLE
80-63 03L-0	10/17/80	11/14/80	UNSAMPLED GAS WASTE RELEASE. LCO 4.8.1

SUMMARY

INSPECTION SUMMARY

* INSPECTION ON OCTOBER 20-23, 1980 (80-20): ROUTINE, UNANNOUNCED INSPECTION OF QUALITY ASSURANCE PROGRAM, TRAINING, REQUALIFICATION TRAINING, AND FOLLOWUP ON PREVIOUSLY IDENTIFIED ITEMS. WITHIN THE FOUR (4) AREAS INSPECTED, ONE ITEM OF NONCOMPLIANCE (EQUIPMENT CONTROL PROCEDURES) WAS IDENTIFIED.

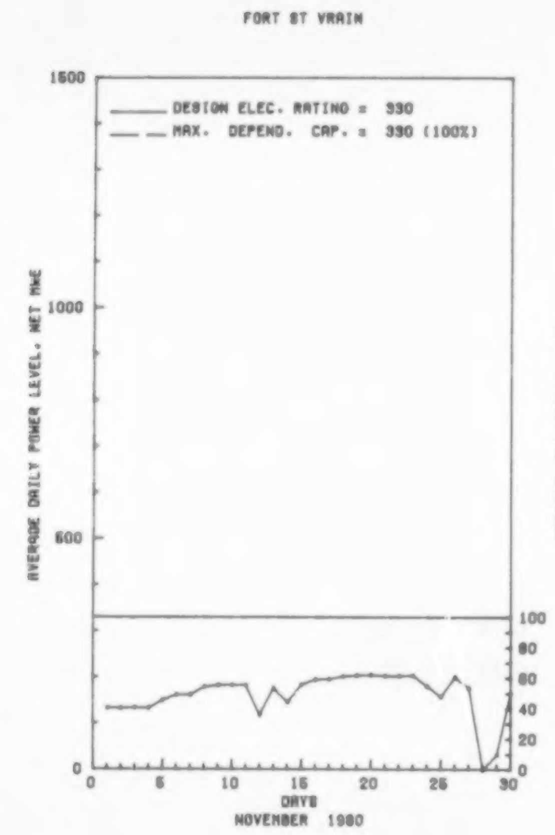
INSPECTION ON OCTOBER 1 - 31, 1980 (80-21): ROUTINE, ANNOUNCED INSPECTION OF REVIEW OF PLANT OPERATIONS; FOLLOWUP ON PREVIOUS ITEMS OF NONCOMPLIANCE; SURVEILLANCE, MAINTENANCE; OPERATIONAL SAFETY, VERIFICATION; REVIEW OF PERIODIC AND SPECIAL REPORTS; AND REVIEW OF LICENSEE EVENT REPORTS. WITHIN THE SEVEN (7) AREAS INSPECTED, ONE ITEM OF NONCOMPLIANCE WAS IDENTIFIED (FAILURE TO COMPLY WITH PROCEDURAL REQUIREMENTS).

REVIEWED BY OMPA LOEB: PE Bole **DATE:** 1/16/81

REVIEWED BY NRR DL: George Kuzmycz **DATE:** 1/16/81

FORT ST. VRAIN

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-267
2. Utility Contact: J. W. GAHM (303) 785-2253
3. Licensed Thermal Power (MWt): 842
4. Nameplate Rating (Gross MWe): 403 x 0.85 = 343
5. Design Electrical Rating (Net MWe): 330
6. Maximum Dependable Capacity (Gross MWe): 342
7. Maximum Dependable Capacity (Net MWe): 330
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): 231
10. Reasons for Restrictions, If Any: NRC RESTRICT 70% PEND. RESOL. OF TEMP. FLUCTUATIONS.

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	8,040.0	12,457.0
12. Hours Reactor Was Critical	720.0	5,877.0	8,439.8
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	683.3	4,052.9	5,035.1
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	352,068	1,871,343	2,349,298
17. Gross Electrical Energy (MWH)	123,96	603,429	744,225
18. Net Electrical Energy (MWH)	117,119	555,744	679,328
19. Unit Service Factor	94.9	50.4	40.4
20. Unit Availability Factor	94.9	50.4	40.4
21. Unit Capacity Factor (MDC Net)	49.3	20.9	16.5
22. Unit Capacity Factor (DER Net)	49.3	20.9	16.5
23. Unit Forced Outage Rate	5.1	26.8	39.7

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): MAINTENANCE & SURVEILLANCE TESTING - MARCH, 1981 3 WEEKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-23	11/12/80	S	20.0	B	5				POWER REDUCTION FOR FLUCTUATION TESTING.
80-24	11/13/80	S	12.0	B	5				POWER REDUCTION FOR FLUCTUATION TESTING.
80-25	11/24/80	F	17.0	A	5		CB	VALVEX	POWER REDUCTION FOR SV-2105 REPAIR.
80-26	11/27/80	F	36.7	A	1		HB	VALVEX	TURBINE GENERATOR TAKEN OFF-LINE TO REPAIR HOT REHEAT DRAIN LINE.

SUMMARY

FORT ST. VRAIN OPERATED WITH 2 REDUCTIONS FOR TESTING, 1 OUTAGE FOR EQUIPMENT FAILURE, AND 1 REDUCTION FOR REPAIR.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

GINNA

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	New York/Wayne 15 miles NE of Rochester
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	1520
5.	Design Electrical Rating (MWE Net)	470
6.	Date of Initial Criticality	November 8, 1969
7.	Date of Commercial Operation	July 1970
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Lake Ontario

Utility & Contractor Information

10.	Licensee	Rochester Gas & Electric Corporation
11.	Corporate Address	89 East Avenue Rochester, New York
12.	Corporate Contact	John Maier, Vice President Electric and Steam Production
13.	Architect/Engineer	Gilbert
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Section Chief	H. B. KISTER
19.	IE Resident Inspector	R. Zimmerman (315) 524-6935 FTS: 264-3311
20.	Licensing Project Manager	R. P. Snaider
21.	Docket Number	50-244
22.	License Number/Date of Issue	DPR-18, 9/19/69
23.	Public Document Room	Rochester Public Library Business and Social Science Division 115 South Avenue Rochester, New York 14604

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

```
    INSPECTION STATUS

    OTHER ITEMS OF INTEREST

    FACILITY ITEMS (PLANS AND PROCEDURES):

*   CRITICALITY IS TENTATIVELY SCHEDULED
    FOR DECEMBER 2, 1980.

    PLANT STATUS:

*   IN COLD SHUTDOWN FOR SCHEDULED MAINTENANCE
    OUTAGE.  MAJOR WORK INCLUDED B STEAM GENERATOR
    TUBE SLEEVING, A & B STEAM GENERATOR CREVICE
    CLEANING, FIRE PROTECTION MODIFICATIONS AND
    SEISMIC SUPPORT MODIFICATIONS.

* LAST IE SITE INSPECTION DATE:  11/1-30/80

*        INSPECTION REPORT NO:  50-244/80-14

    SUMMARY

    INSPECTION SUMMARY

*   NONE
```

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

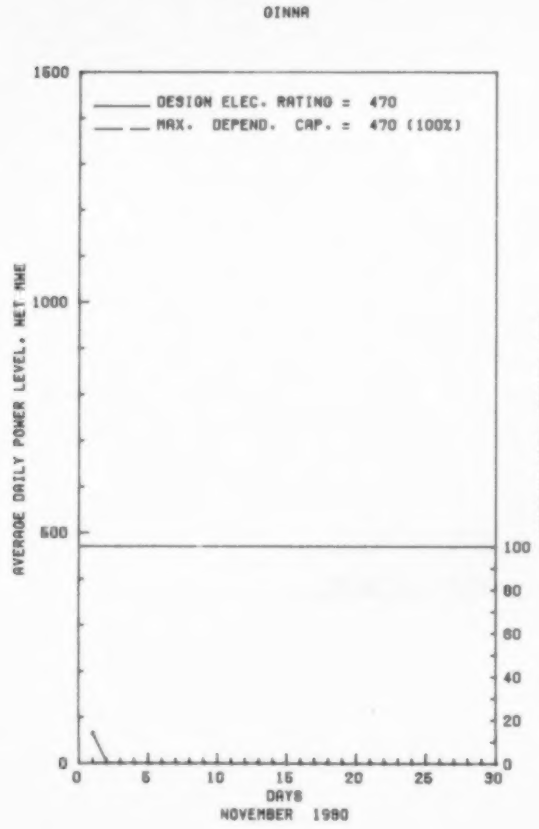
```
==========================================================================================
 NUMBER    DATE OF     DATE OF     SUBJECT
            EVENT       REPORT
==========================================================================================
 NONE
==========================================================================================
```

REVIEWED BY OMPA LOEB: PE Bobe **DATE:** 1/16/81

REVIEWED BY NRR DL: **DATE:** 1/16/81

GINNA

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-244
2. Utility Contact: A.E. MCNAMARA (315) 524-4446
3. Licensed Thermal Power (MWt): 1520
4. Nameplate Rating (Gross MWe): 608 x 0.85 = 517
5. Design Electrical Rating (Net MWe): 470
6. Maximum Dependable Capacity (Gross MWe): 490
7. Maximum Dependable Capacity (Net MWe): 470

CAPACITY FACTORS CALCULATED WITH A WEIGHTED AVEPAGE

8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	8,040.0	96,552.0
12. Hours Reactor Was Critical	.8	6,019.8	73,699.6
13. Reactor Reserve Shutdown Hours	.0	.0	1,621.8
14. Hours Generator On-Line	.2	5,980.5	71,937.[illegible]
15. Unit Reserve Shutdown Hours	.0	.0	8.5
16. Gross Thermal Energy (MWH)	72	8,890,848	97,166,650
17. Gross Electrical Energy (MWH)	16	2,911,845	31,597,636
18. Net Electrical Energy (MWH)	11	2,768,396	29,929,916
19. Unit Service Factor	.0	74.4	74.5
20. Unit Availability Factor	.0	74.4	74.5
21. Unit Capacity Factor (MDC Net)	.0	73.3	68.2
22. Unit Capacity Factor (DER Net)	.0	73.3	68.2
23. Unit Forced Outage Rate	.0	.0	9.2

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 12/02/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
2	11/01/80	S	719.8	B	1		CC	HTEXCH	STEAM GENERATORS INSPECTION

SUMMARY

THE UNIT SHUTDOWN AT THE BEGINNING OF THE MONTH FOR STEAM GENERATOR INSPECTION. PREVIOUS EDDY CURRENT TESTING HAD INDICATED INTERGRANNULAR ATTACK IN THE TUBE CREVICE AREA.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

HADDAM NECK

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Connecticut/Middlesex 13 miles E of Meriden
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	1825
5.	Design Electrical Rating (MWE Net)	580
6.	Date of Initial Criticality	July 24, 1967
7.	Date of Commercial Operation	January 1, 1968
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Connecticut River

Utility & Contractor Information

10.	Licensee	Connecticut Yankee Atomic Power Co.
11.	Corporate Address	P. O. Box 270 Hartford, Connecticut
12.	Corporate Contact	D. Switzer, President
13.	Architect/Engineer	Stone & Webster
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Section Chief	R. Keimig
19.	IE Resident Inspector	T. H. Smith (203) 267-2571,2
20.	Licensing Project Manager	R. Caruso
21.	Docket Number	50-213
22.	License Number/Date of Issue	DPR-61, 12/27/74
23.	Public Document Room	Russell Library 119 Broad Street Middletown, Connecticut 06457

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

```
   INSPECTION STATUS

   OTHER ITEMS OF INTEREST

   PLANT STATUS:

+  ROUTINE POWER OPERATION.

+ LAST IE SITE INSPECTION DATE:  11/19-21/80

+        INSPECTION REPORT NO:  50-213/80-24

   SUMMARY

   INSPECTION SUMMARY

+  50-213/80-19 - OCT 6-10:  ROUTINE, UNANNOUNCED PHYSICAL SECURITY
   INSPECTION BY TWO REGION-BASED INSPECTORS (64 HOURS) OF:  SECURITY
   PLAN; SECURITY ORGANIZATION; PHYSICAL BARRIERS (PROTECTED AREA);
   PHYSICAL BARRIERS (VITAL AREAS); LIGHTING; ACCESS CONTROL (IDENTI-
   FICATION, AUTHORIZATION, BADGING); ACCESS CONTROL (SEARCH), ACCESS
   CONTROL (ESCORTING); ACCESS CONTROL (VITAL AREAS); DETECTION AIDS;
   ALARM STATIONS; ASSESSMENT AIDS; COMMUNICATIONS; TESTING AND
   MAINTENANCE; COMPENSATORY MEASURES; POWER SUPPLY; RESPONSE; LOCKS,
   KEYS AND COMBINATIONS; AND RECORDS AND REPORTS.  NO NONCOMPLIANCES
   WERE IDENTIFIED.
```

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

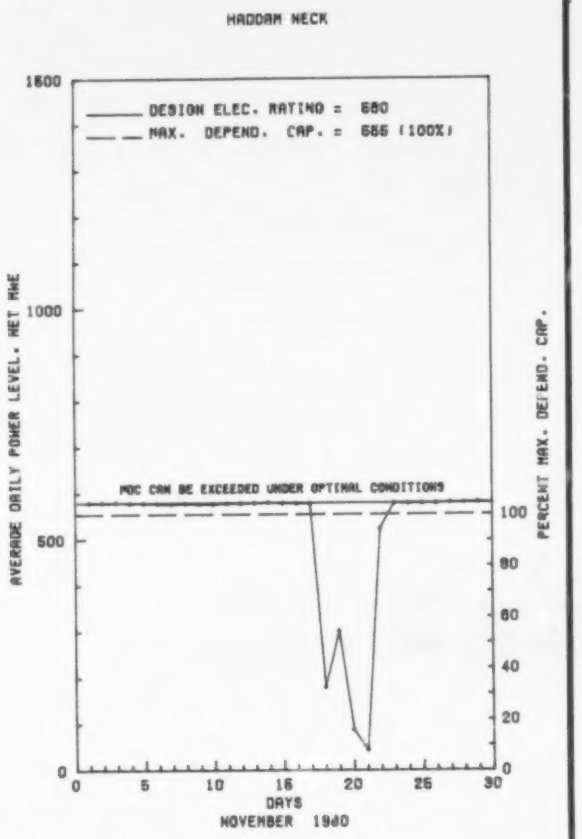
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE			

REVIEWED BY OMPA LOEB: PE Rola **DATE:** 1/16/81

REVIEWED BY NRR DL: **DATE:** 1/16/81

HADDAM NECK

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

CAPACITY FACTORS CALCULATED WITH A WEIGHTED AVERAGE

1. Docket Number: 50-213
2. Utility Contact: DON ANDERSON (203) 267-2556 X274
3. Licensed Thermal Power (MWt): 1825
4. Nameplate Rating (Gross MWe): 667 x 0.9 = 600
5. Design Electrical Rating (Net MWe): 580
6. Maximum Dependable Capacity (Gross MWe): 577
7. Maximum Dependable Capacity (Net MWe): 555
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	8,040.0	113,232.0
12. Hours Reactor Was Critical	708.0	6,031.9	97,228.6
13. Reactor Reserve Shutdown Hours	.0	27.0	1,179.6
14. Hours Generator On-Line	672.7	5,840.6	92,780.8
15. Unit Reserve Shutdown Hours	.0	.0	369.9
16. Gross Thermal Energy (MWH)	1,184,795	10,981,831	160,222,980
17. Gross Electrical Energy (MWH)	392,585	3,307,160	52,661,626
18. Net Electrical Energy (MWH)	373,847	3,141,191	50,095,954
19. Unit Service Factor	93.4	72.6	81.9
20. Unit Availability Factor	93.4	72.6	82.3
21. Unit Capacity Factor (MDC Net)	93.6	70.4	81.7
22. Unit Capacity Factor (DER Net)	89.5	67.4	78.5
23. Unit Forced Outage Rate	6.5	1.3	6.9

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each). NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-7	11/18/80	F	13.8	A	2	80-16/03L	RB	CRDRVE	WHILE STEPPING IN CONTROL BANK RODS, ONE STEP, BOTH AUDIBLE AND VISUAL ALARMS INDICATED TWO DROPPED RODS-MOVABLE GRIPPER COILS WERE AT FAULT
80-8	11/20/80	F	32.9	H	3		ZZ	ZZZZZZ	IT WAS DETERMINED THAT THE MECHANICAL OVERSPEED DEVICE ON THE H.P. TURBINE WAS OUT OF ADJUSTMENT.
80-9	11/21/80	S	.7	B	1		ZZ	ZZZZZZ	TEST OF OVERSPEED SETTING ON TURBINE

SUMMARY

THE UNIT OPERATED ROUTINELY EXCEPT FOR THE THREE OUTAGES LISTED IN DETAIL ABOVE

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

HATCH 1

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Georgia/Appling 11 miles N of Baxley
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2436
5.	Design Electrical Rating (MWE Net)	777
6.	Date of Initial Criticality	September 12, 1974
7.	Date of Commercial Operation	December 31, 1975
8.	Condenser Cooling Method	Once Through With Forced Draft Cooling Towers
9.	Condenser Cooling Water Source	Altamaha River

Utility & Contractor Information

10.	Licensee	Georgia Power Company
11.	Corporate Address	270 Peachtree Street, NE Atlanta, Georgia 30303
12.	Corporate Contact	J. H. Miller Executive Vice President
13.	Architect/Engineer	Southern Services, Inc.
14.	Nuclear Steam Supply System	GE
15.	Constructor	Georgia Power Company

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. Lewis
18.	IE Section Chief	H. C. Dance
19.	IE Resident Inspector	R. F. Rogers (912) 367-9881
20.	Licensing Project Manager	D. Verrelli
21.	Docket Number	50-321
22.	License Number/Date of Issue	DPR-57, 8/6/74
23.	Public Document Room	Appling County Public Library Parker Street Baxley, Georgia 31513

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

PLANT IS OPERATING AT REDUCED POWER.

+ LAST IE SITE INSPECTION DATE: NOVEMBER 4-7, 1980

+ INSPECTION REPORT NO: 50-321/80-43

SUMMARY

INSPECTION SUMMARY

+ INSPECTION NOVEMBER 4-7 (80-43): THIS SPECIAL, ANNOUNCED INSPECTION INVOLVED 13 INSPECTOR-HOURS ON SITE IN THE AREAS OF INSTALLED EQUIPMENT REVIEW WITH RESPECT TO THE IE BULLETIN 79-01B RESPONSE. OF THE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATION WERE IDENTIFIED.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

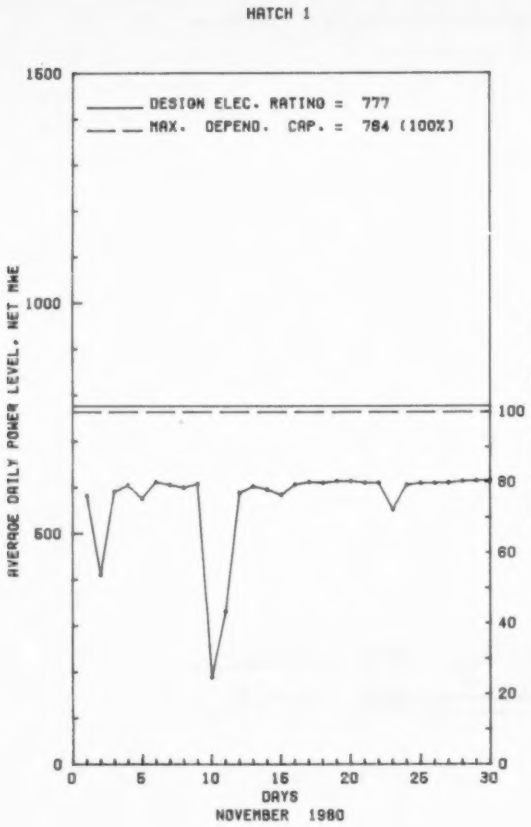
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-106/ 03L-0	10/22/80	11/11/80	CHLORINE MONITOR INOPERATIVE FOR CONTROL ROOM
80-112/ 03L-0	10/23/80	11/11/80	DOOR CLOSER ON 4 FIRE RATED DOOR INOPERATIVE

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 1-16-81

REVIEWED BY NRR DL: **DATE:** 1-17-81

HATCH 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-321
2. Utility Contact: P. R. UNDERWOOD (912) 367-7781
3. Licensed Thermal Power (MWt): 2436
4. Nameplate Rating (Gross MWe): 1000 x 0.85 = 850
5. Design Electrical Rating (Net MWe): 777
6. Maximum Dependable Capacity (Gross MWe): 797
7. Maximum Dependable Capacity (Net MWe): 764
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): 640
10. Reasons for Restrictions, If Any: MANAGEMENT LIMITATION

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	8,040.0	43,104.0
12. Hours Reactor Was Critical	708.0	7,016.4	32,814.4
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	700.8	6,529.9	30,798.3
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,324,189	14,288,551	64,641,586
17. Gross Electrical Energy (MWH)	432,870	4,656,600	20,927,540
18. Net Electrical Energy (MWH)	412,803	4,437,629	19,899,562
19. Unit Service Factor	97.3	81.2	71.5
20. Unit Availability Factor	97.3	81.2	71.5
21. Unit Capacity Factor (MDC Net)	75.0	72.2	60.4
22. Unit Capacity Factor (DER Net)	73.8	71.0	59.4
23. Unit Forced Outage Rate	2.7	18.8	16.7

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUELING, MARCH 1, 1981; 13 WEEKS
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-66	11/01/80	S	21.8	F	5		ZZ	ZZZZZZ	MANAGEMENT LIMITATION OF 80% THERMAL POWER
80-67	11/01/80	S	34.3	F	5		RB	CONROD	REDUCED LOAD TO PERFORM A ROD SEQUENCE EXCHANGE
80-68	11/03/80	S	56.2	F	5		ZZ	ZZZZZZ	MANAGEMENT LIMITATION OF 80% THERMAL POWER
80-69	11/05/80	F	5.3	H	5	80-116	CB	PUMPXX	LOAD REDUCED BECAUSE OF "A" RECIRC PUMP TRIP
80-70	11/05/80	S	75.1	F	5		ZZ	ZZZZZZ	MANAGEMENT LIMITATION OF 80% THERMAL POWER
80-71	11/09/80	S	1.2	F	5		HE	TURBIN	REDUCED LOAD TO PERFORM WEEKLY TURBINE TEST
80-72	11/09/80	S	30.0	F	5		ZZ	ZZZZZZ	MANAGEMENT LIMITATION OF 80% THERMAL POWER
80-73	11/10/80	F	19.2	B	3		CD	VALVEX	RX AUTO SCRAM DUE TO MSIV CLOSURE WHILE PERFORMING HNP-1-SRV-03005
80-74	11/10/80	F	28.0	B	5		CD	VALVEX	UNIT ON STARTUP RAMP FROM ABOVE SHUTDOWN, #80-73
80-75	11/12/80	S	63.3	F	5		ZZ	ZZZZZZ	MANAGEMENT LIMITATION OF 80% THERMAL POWER
80-76	11/14/80	S	6.7	F	5		RB	CONROD	REDUCED LOAD IN ORDER TO PERFORM A ROD PATTERN ADJUSTMENT AND A WEEKLY TURBINE TEST
80-77	11/15/80	S	185.2	F	5		ZZ	ZZZZZZ	MANAGEMENT LIMITATION OF 80% THERMAL POWER
80-78	11/22/80	S	7.8	F	5		RB	CONROD	REDUCED LOAD IN ORDER TO PERFORM A ROD PATTERN ADJUSTMENT AND A WEEKLY TURBINE TEST
80-79	11/23/80	S	138.6	F	5		ZZ	ZZZZZZ	MANAGEMENT LIMITATION OF 80% THERMAL POWER
80-80	11/29/80	S	3.5	F	5		HE	TURBIN	REDUCED LOAD TO PERFORM DELTA WEEKLY TURBINE TEST
80-81	11/29/80	S	43.8	F	5		ZZ	ZZZZZZ	MANAGEMENT LIMITATION OF 80% THERMAL POWER

SUMMARY

THE UNIT WAS LIMITED TO 80% POWER BY THE UTILITY DURING THE MONTH.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

HATCH 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1. Location (state/county) (nearest population center): Georgia/Appling, 11 miles N of Baxley
2. Electric Reliability Council: Southeastern Electric Reliability Council
3. Type of Reactor: BWR
4. Power Level (MWT): 2436
5. Design Electrical Rating (MWE Net): 784
6. Date of Initial Criticality: July 4, 1978
7. Date of Commercial Operation: September 5, 1979
8. Condenser Cooling Method: Cooling Towers
9. Condenser Cooling Water Source: Altamaha River

Utility & Contractor Information

10. Licensee: Georgia Power Company
11. Corporate Address: 270 Peach Street, NE, Atlanta, Georgia 30303
12. Corporate Contact: J. H. Miller, Executive Vice President
13. Architect/Engineer: Southern Services, Inc./Bechtel
14. Nuclear Steam Supply System: GE
15. Constructor: Georgia Power Company

Regulatory Information

16. IE Region responsible: II
17. IE Branch Chief: R. C. Lewis
18. IE Section Chief: H. C. Dance
19. IE Resident Inspector: R. F. Rogers (912) 367-9881
20. Licensing Project Manager: D. Verrelli
21. Docket Number: 50-366
22. License Number/Date of Issue: NPF-5/June 13, 1978
23. Public Document Room: Appling County Public Library, Parker Street, Baxley, Georgia 31513

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

REACTOR IS IN COLD SHUTDOWN FOR REFUELING.

- LAST IE SITE INSPECTION DATE: NOVEMBER 4-7, 1980
- INSPECTION REPORT NO: 50-366/80-43

SUMMARY

INSPECTION SUMMARY

- INSPECTION NOVEMBER 4-7 (80-43): THIS SPECIAL, ANNOUNCED INSPECTION INVOLVED 13 INSPECTOR-HOURS ON SITE IN THE AREAS OF INSTALLED EQUIPMENT REVIEW WITH RESPECT TO THE IE BULLETIN 79-01B RESPONSE. OF THE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATION WERE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

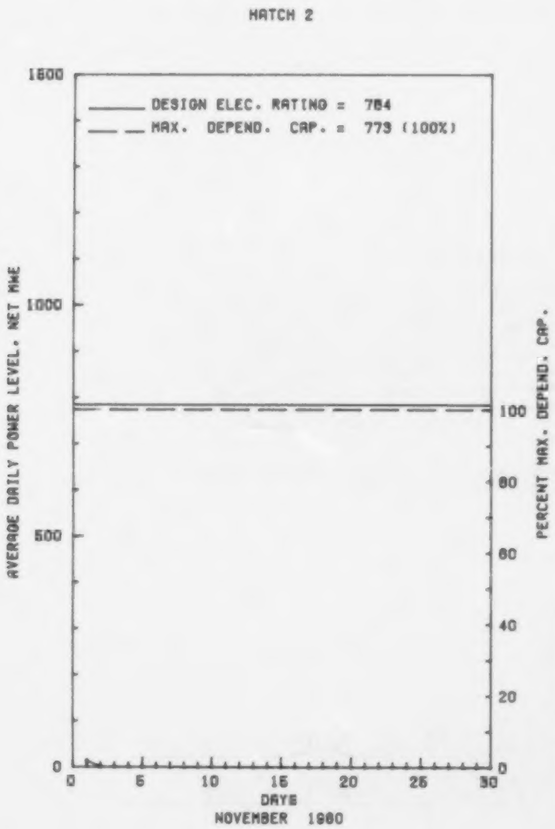
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-140/ 03L-0	10/09/80	10/23/80	DRYWELL VACUUM BREAKER TEST MISSED
80-141/ 03L-0	10/14/80	11/04/80	HIGH PRESSURE COOLANT INJECTION STEM LINE PRESSURE SWITCH SETPOINT DRIFT
80-143/ 03L-0	10/15/80	11/04/80	MAIN STEAM LINE PRESSURE SWITCH TRIPPED LOW OUT OF SPECIFICATION
80-148/ 03L-0	11/01/80	11/13/80	TORUS WATER LEVEL DECREASED BELOW LIMIT

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 1-16-81

REVIEWED BY NRR DL: **DATE:** 1-17-81

HATCH 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-366
2. Utility Contact: P. R. UNDERWOOD (912) 367-7781
3. Licensed Thermal Power (MWt): 2436
4. Nameplate Rating (Gross MWe): 1000 x .85 = 850
5. Design Electrical Rating (Net MWe): 784
6. Maximum Dependable Capacity (Gross MWe): 806
7. Maximum Dependable Capacity (Net MWe): 773
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	8,040.0	10,873.0
12. Hours Reactor Was Critical	3.1	5,570.5	8,234.0
13. Reactor Reserve Shutdown Hours	.0	0	.0
14. Hours Generator On-Line	2.8	5,273.7	7,688.4
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	2,822	11,603,836	17,092,898
17. Gross Electrical Energy (MWH)	670	3,831,320	5,667,280
18. Net Electrical Energy (MWH)	-3,009	3,647,389	5,406,526
19. Unit Service Factor	.4	65.6	70.7
20. Unit Availability Factor	.4	65.6	70.7
21. Unit Capacity Factor (MDC Net)	.0	58.7	64.3
22. Unit Capacity Factor (DER Net)	.0	57.9	63.4
23. Unit Forced Outage Rate	.0	12.4	10.9

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 02/05/81

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-52	11/31/80	S	2.8	C	5		ZZ	ZZZZZZ	REDUCED LOAD TO SHUTDOWN FOR REFUEL - OUTAGE
80-53	11/01/80	S	717.2	C	2		ZZ	ZZZZZZ	SHUTDOWN FOR REFUELING OUTAGE.

SUMMARY

THE UNIT ENTERED INTO ITS FIRST REFUELING OUTAGE THIS MONTH.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

HUMBOLDT BAY

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	California/Humboldt 4 miles S of Eureka
2.	Electric Reliability Council	Western Systems Coordinating Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	220
5.	Design Electrical Rating (MWE Net)	65
6.	Date of Initial Criticality	February 16, 1963
7.	Date of Commercial Operation	August 1963
8.	Condenser Cooling Method	Circulating sea water
9.	Condenser Cooling Water Source	Pacific Ocean (Humboldt Bay)

Utility & Contractor Information

10.	Licensee	Pacific Gas & Electric Co.
11.	Corporate Address	77 Beale Street San Francisco, California 94106
12.	Corporate Contact	P. A. Crane, Jr. Asst. General Counsel
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	GE
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	V
17.	IE Branch Chief	J. L. Crews
18.	IE Section Chief	D. M. Sternberg
19.	IE Resident Inspector	J. D. Carlson
20.	Licensing Project Manager	V. L. Rooney
21.	Docket Number	50-133
22.	License Number/Date of Issue	DPR-7, 8/28/62
23.	Public Document Room	Humboldt County Library 636 F Street Eureka, California 95501

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

THE PLANT HAS BEEN SHUT DOWN SINCE JULY 1976.

ITEMS REMAINING TO BE COMPLETED DURING THE OUTAGE HAVE BEEN DEFERRED PENDING RESOLUTION OF THE SEISMIC ISSUES AT THE PLANT SITE. A DATE FOR RETURN TO OPERATIONS HAS NOT BEEN ESTABLISHED.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

PLANT IS SHUTDOWN.

LAST IE SITE INSPECTION DATE: 11/10/80+

INSPECTION REPORT NO: 50-133/80-03+

SUMMARY

INSPECTION SUMMARY

+ INSPECTION ON NOVEMBER 10, 1980 (REPORT NO. 50-133/80-03) AREAS INSPECTED: SPECIAL INSPECTION TO EXAMINE EFFECTS OF EARTHQUAKE ON NOVEMBER 8, 1980. THE INSPECTION INVOLVED FIVE INSPECTOR HOURS ONSITE BY ONE NRC INSPECTOR.

RESULTS: NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE			

LICENSE STATUS

On December 31, 1980, Pacific Gas and Electric Company (PG&E) filed a motion to withdraw its application for license amendment to permit the restart of the Humboldt Bay Power Plant, Unit No. 3. The licensee cited the high potential costs of additional equipment and operating costs when measured against the size of the facility and its remaining useful life. PG&E also noted that further definition of the NRC backfit requirement policy may have a substantial impact on these costs and that when NRC retrofit standards become better known, currently projected costs may be less than expected and it may then be economic to make the required modifications.

The utility plans to evaluate the evolving NRC backfit requirements as well as other alternatives including decommissioning.

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 1-16-81

REVIEWED BY NRR DL: V Rooney **DATE:** 1/16/81

HUMBOLDT BAY

AVERAGE DAILY POWER LEVEL (MWe)

OPERATING STATUS

1. Docket Number: 50-133
2. Utility Data Prepared By: B. C. Getty (707) 443-2787
3. Licensed Thermal Power (MWt): 220
4. Nameplate Rating (Gross MWe): 77 X 0.85 = 65
5. Design Electrical Rating (Net MWe): 65
6. Maximum Dependable Capacity (Gross MWe): 0
7. Maximum Dependable Capacity (Net MWe): 0
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons:
None

9. Power Level To Which Restricted, If Any (Net MWe): None
10. Reasons For Restrictions, If Any:

This ... ive

11. Hours In Reporting Period
12. Number Of Hours Reactor Was Cri
13. Reactor Reserve Shutdow
14. Hours Generator
15. Unit R
rt)
erated (MWH)
Net gy Generated (MWH)
19. Unit S ce Factor
20. Unit Availability Factor
21. Unit Capacity Factor (Using MDC Net) 1/
22. Unit Capacity Factor (Using DER Net) 1/
23. Unit Forced Outage Rate
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
None

25. If Shut Down At End Of Report Period, Estimated Date of Startup: UNKNOWN

NO DECISION ON FUTURE OPERATION
SHUTDOWN INDEFINITELY

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
SEE SUMMARY BELOW									

SUMMARY

THE UNIT WAS SHUTDOWN ON 7/2/76 FOR SEISMIC MODIFICATIONS. THE UNIT REMAINS IN THE COLD SHUTDOWN CONDITION.

LEGEND

[1]Type:
F–Forced
S–Scheduled

[2]Reason:
A–Equipment Failure
B–Maintenance or Test
C–Refueling
D–Regulatory Restriction
E–Operator Training & License Examination
F–Administrative
G–Operational Error
H–Other

[3]Method:
1–Manual
2–Manual Scram
3–Automatic Scram
4–Continuations
5–Load Reduction
9–Other

[4]Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5]Exhibit H:
Same source.

INDIAN POINT 2

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	New York/Westchester 25 miles N of New York City
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2758
5.	Design Electrical Rating (MWE Net)	873
6.	Date of Initial Criticality	May 22, 1973
7.	Date of Commercial Operation	August 1973
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Hudson River

Utility & Contractor Information

10.	Licensee	Consolidated Edison Company of New York, Inc.
11.	Corporate Address	4 Irving Place New York, New York
12.	Corporate Contact	J. O'Toole, Asst. to Vice President
13.	Architect/Engineer	UE&C
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	J. A. Jones/UE&C

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Senior Inspectc	H. B. KISTER
19.	IE Resident Inspector	T. Rebelowski (914-739-9360) FTS: 264-3311
20.	Licensing Project Manager	L. Olshan (X27144)
21.	Docket Number	50-247
22.	License Number/Date of Issue	DPR-26/9-28-73
23.	Public Document Room	White Plains Public Library 100 Martine Avenue White Plains, New York 10601

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

- THE LICENSEE IS IN THE PROCESS OF IDENTIFYING CONCERNS OF IMMEDIATE ACTION LETTER 80-41 IN RELATION TO THE SHUTDOWN OF OCTOBER 17, 1980 AND THE FLOODING OF THE VAPOR CONTAINMENT BUILDING.

FACILITY ITEMS (PLANS AND PROCEDURES):

- THE LICENSEE DECIDED TO BEGIN A SCHEDULED REFUELING OUTAGE EARLY TO FACILITATE REPLACEMENT OF THE FAN COOLER UNITS.

PLANT STATUS:

- UNIT IS SHUTDOWN.

- LAST IE SITE INSPECTION DATE: 11/23/80
- INSPECTION REPORT NO: 50-247/80-21

SUMMARY

INSPECTION SUMMARY

- 50-247/80-10 - JUL 21 - AUG 8: ROUTINE, UNANNOUNCED ONSITE INSPECTION BY TWO REGION-BASED INSPECTOR (32 HRS) OF ENVIRONMENTAL MONITORING PROGRAMS INCLUDING: THE MANAGEMENT CONTROLS OF THESE PROGRAMS; THE LICENSEE PROGRAM FOR QUALITY CONTROL OF ANALYTICAL MEASUREMENTS; IMPLEMENTATION OF ENVIRONMENTAL MONITORING PROGRAMS - RADIOLOGICAL; IMPLEMENTATION OF ENVIRONMENTAL MONITORING PROGRAMS - BIOLOGICAL/ECOLOGICAL; NONRADIOACTIVE RELEASE RATES AND LIMITS; AND A FOLLOWUP ON THE LICENSEE ACTION ON PREVIOUS INSPECTION FINDINGS. THREE NONCOMPLIANCES WERE IDENTIFIED: FAILURE TO PERFORM SR-90 ANALYSES OF DRINKING WATER AND LAKE WATER SAMPLES DURING 1979; FAILURE TO COLLECT AND ANALYZE DRINKING WATER SAMPLES; FAILURE TO RECORD THE RATE OF DELTA TC DURING ALL FLOW CONDITIONS.

- 50-247/80-14 - SEP 3-4: SPECIAL, ANNOUNCED, ONSITE INSPECTION BY ONE REGION-BASED INSPECTOR (6 HRS) OF THE METEOROLOGICAL MONITORING SYSTEM. THE INSPECTOR EXAMINED THE MONITORING EQUIPMENT AND THE SUPPORT FACILITIES IN THE EMERGENCY OPERATIONS FACILITY AND REVIEWED LICENSEE ACTIONS TAKEN IN RESPONSE TO THE FEBRUARY 11, 1980 CONFIRMATORY ORDER IN THE FOLLOWING AREAS: PRIMARY METEOROLOGICAL MEASUREMENTS PROGRAM; BACKUP METEOROLOGICAL MEASUREMENTS PROGRAM; REAL-TIME PREDICTIONS OF ATMOSPERHIC EFFLUENT TRANSPORT AND DIFFUSION; AND REMOTE INTERROGATION OF THE ATMOSPHERIC MEASUREMENT AND PREDICTION SYSTEMS. NO NONCOMPLIANCES WERE IDENTIFIED.

- 50-247/80-18 - OCT 6-10: ROUTINE, UNANNOUNCED PHYSICAL SECURITY INSPECTION BY ONE REGION-BASED INSPECTOR (42 HRS) INCLUDING: IMPLEMENTATION AND COMPLIANCE WITH THE ACCEPTED PHYSICAL SECURITY PLAN RELATIVE TO THE SECURITY PROGRAM AUDIT, SECURITY ORGANIZATION, PHYSICAL BARRIERS (PA)(VA), LIGHTING, ACCESS CONTROLS, (IDENTIFICATION), (SEARCH), (ESCORTING), AND (VITAL AREAS), ALARM STATIONS, DETECTION AIDS, ASSESSMENT AIDS, COMMUNICATIONS, TESTING AND MAINTENANCE, COMPENSATORY MEASURES, POWER SUPPLY, RESPONSE, LOCKS, KEYS AND COMBINATIONS, AND RECORDS AND REPORTS, I&E BULLETIN 79-16, AND I&E CIRCULAR 80-09. FOLLOWUP ON LICENSEE ACTION ON PREVIOUSLY IDENTIFIED ITEMS OF NONCOMPLIANCE RELATED TO PHYSICAL SECURITY. THREE NONCOMPLIANCES WERE IDENTIFIED: FAILURE TO TRAIN A WATCHPERSON PRIOR TO ASSIGNMENT; FAILURE TO MAINTAIN THE ISOLATION ZONE FREE OF OBSTRUCTIONS; AND FAILURE TO INSURE THAT THE SHIFT SUPERVISOR KEEP MAINTENANCE AND REPAIR RECORDS.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

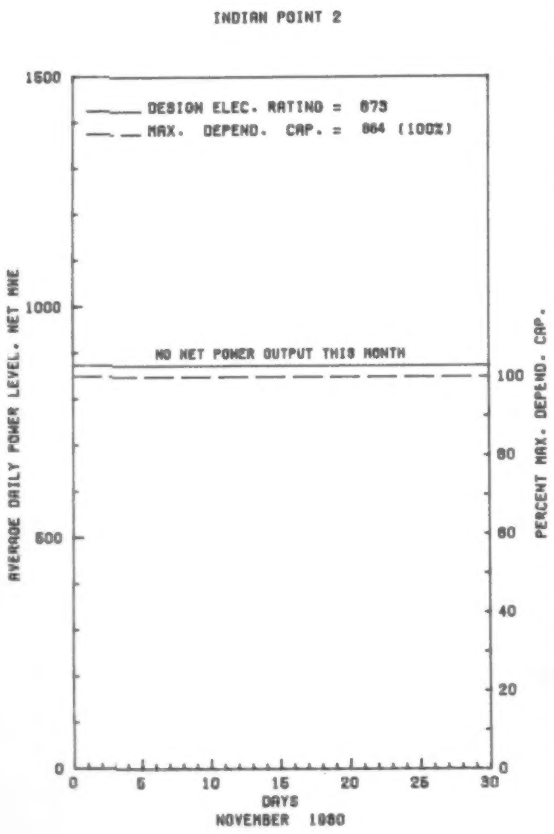
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-012/01T	10/16/80	10/29/80	SODIUM HYDROXIDE VALVE OUT OF SERVICE DURING SURVEILLANCE TESTING
80-015/03L	10/17/80	11/14/80	FAILURE OF N42 DUE TO MOISTURE IN ELECTRICAL CONNECTOR
80-014/03L	10/17/80	11/14/80	MS-1 MAIN STEAM ISOLATION VALVE FAILED TO COMPLETELY CLOSE DURING A SHUTDOWN OF SECONDARY PLANT

REVIEWED BY OMPA LOEB: ________ **DATE:** 1/16/81

REVIEWED BY NRR DL: ________ **DATE:** ________

INDIAN POINT 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-247
2. Utility Contact: E. EICH (914) 694-6000 @ I.P.
3. Licensed Thermal Power (MWt): 2758
4. Nameplate Rating (Gross MWe): 1126 x 0.9 = 1013
5. Design Electrical Rating (Net MWe): 873
6. Maximum Dependable Capacity (Gross MWe): 900
7. Maximum Dependable Capacity (Net MWe): 864
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): 839
10. Reasons for Restrictions, If Any: REMOV. OF NO.3 DISC. (GENER. END) ON NO.23 L.P. TURB. ROTOR

ITEM 21 CALCULATED BY USING A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE *
11. Hours in Reporting Period	720.0	8,040.0	56,281.0
12. Hours Reactor Was Critical	.0	5,869.4	38,163.9
13. Reactor Reserve Shutdown Hours	.0	1,231.2	1,472.6
14. Hours Generator On-Line	.0	5,691.2	37,080.4
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	14,975,902	95,808,428
17. Gross Electrical Energy (MWH)	0	4,486,960	29,620,046
18. Net Electrical Energy (MWH)	-7,494	4,266,232	28,231,332
19. Unit Service Factor	.0	70.8	65.9
20. Unit Availability Factor	.0	70.8	65.9
21. Unit Capacity Factor (MDC Net)	.0	62.0	58.3
22. Unit Capacity Factor (DER Net)	.0	60.8	57.5
23. Unit Forced Outage Rate	100.0	15.3	9.3

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 04/15/81

*COMPUTED SINCE 7/1/74, THE DATE OF COMPLETION OF A 100 DAY – 100% POWER OPERATION TEST.

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
14	10/17/80	F	120.0	A	4		HA	INSTRU	LOCAL TURBINE LOAD LIMIT MOVED IN DECREASING DIRECTION CAUSING SYSTEM TRANSIENT AND RESULTANT REACTOR TRIP VIA HIGH PRESSURIZER PRESSURE SIGNAL.
14	11/06/80	S	600.0	C	9		XX	XXXXXX	OUTAGE CONTINUED TO REPLACE THE FAN COOLER HEAT EXCHANGERS, A REFUELING, MAINTENANCE WORK, AND PLANT MODIFICATIONS.

SUMMARY

UNIT NO. 2 IS CURRENTLY OUT OF SERVICE TO REPLACE THE FAN COOLER HEAT EXCHANGERS, REFUELING, MAINTENANCE WORK, AND PLANT MODIFICATIONS. ALL THREE LOW PRESSURE TURBINES HAVE BEEN DISASSEMBLED FOR INSPECTION AND RESTACKING OF THE ROTOR FOR NO. 23 TURBINE. A REPORT DESCRIBING THE ACCUMULATION OF WATER IN CONTAINMENT IS BEING PREPARED AND WILL BE FORWARDED TO THE COMMISSION UPON COMPLETION.

LEGEND

[1] Type:
F–Forced
S–Scheduled

[2] Reason:
A–Equipment Failure
B–Maintenance or Test
C–Refueling
D–Regulatory Restriction
E–Operator Training & License Examination
F–Administrative
G–Operational Error
H–Other

[3] Method:
1–Manual
2–Manual Scram
3–Automatic Scram
4–Continuations
5–Load Reduction
9–Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

INDIAN POINT 3

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	New York/Westchester 25 miles N of New York City
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	3025
5.	Design Electrical Rating (MWE Net)	965
6.	Date of Initial Criticality	April 6, 1976
7.	Date of Commercial Operation	August 30, 1976
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Hudson River

Utility & Contractor Information

10.	Licensee	Power Authority of the State of New York, Inc.
11.	Corporate Address	10 Columbus Circle New York, New York
12.	Corporate Contact	J. P. Bayne - Senior Vice President.
13.	Architect/Engineer	UE&C
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	WEDCO

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Section Chief	H. B. KISTER
19.	IE Resident Inspector	T. Rebelowski (914-739-9360) FTS: 264-3311 - Operator
20.	Licensing Project Manager	L. Olshan x27144
21.	Docket Number	50-286
22.	License Number/Date of Issue	DPR-64, 12/12/75
23.	Public Document Room	White Plains Public Library 100 Martine Avenue White Plains, New York 10601

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

```
        I N S P E C T I O N   S T A T U S

        OTHER ITEMS OF INTEREST

+       SYSTEMS AND COMPONENT PROBLEMS:

+       MINOR LEAKS IN FAN COOLERS ARE BEING
        REPAIRED PRIOR TO UNIT STARTUP FROM THE
        CURRENT OUTAGE.

        FACILITY ITEMS (PLANS AND PROCEDURES):

+       EXPECTED CRITICALITY ON DECEMBER 5, 1980.

        PLANT STATUS:

+       UNIT IS SHUTDOWN IN AN OUTAGE FOR TURBINE
        EXAMINATION, STEAM GENERATOR EXAMINATION
        AND INSTALLATION OF TMI MODIFICATIONS.

* LAST IE SITE INSPECTION DATE:  11/5-6/80

+           INSPECTION REPORT NO:  50-286/80-18

        SUMMARY

        INSPECTION SUMMARY

+       50-286/80-11 - JUL 21 - AUG 8: ROUTINE, UNANNOUNCED ONSITE
        INSPECTION BY ONE REGION-BASED INSPECTOR (28 HRS) OF ENVIRON-
        MENTAL MONITORING PROGRAMS INCLUDING: THE MANAGEMENT CONTROLS
        OF THESE PROGRAMS; THE LICENSEE PROGRAM FOR QUALITY CONTROL
        OF ANALYTICAL MEASUREMENTS; IMPLEMENTATION OF ENVIRONMENTAL
        MONITORING PROGRAMS - RADIOLOGICAL; IMPLEMENTATION OF ENVIRON-
        MENTAL PROGRAM - BIOLOGICAL/ECOLOGICAL; NONRADIOACTIVE RELEASE
        RATES AND LIMITS; AND A FOLLOWUP ON THE LICENSEE ACTION ON
        PREVIOUS INSPECTION FINDINGS.  FIVE NONCOMPLIANCES WERE IDENT-
        IFIED: FAILURE TO PERFORM SR-90 ANALYSES OF DRINKING WATER AND
        LAKE WATER SAMPLES; FAILURE TO COLLECT AND ANALYZE DRINKING
        WATER SAMPLES; FAILURE TO INCLUDE LAND USE CENSUS DATA IN
        ANNUAL ENVIRONMENTAL OPERATING REPORT; FAILURE TO APPROVE
        TEMPORARY PROCEDURE CHANGES IN ACCORD WITH ETSR; FAILURE TO
        RECORD THE RATE OF DELTA TC DURING ALL FLOW CONDITIONS.

+       50-286/80-14 - SEP 3-4: SPECIAL, ANNOUNCED ONSITE INSPECTION
        BY ONE REGION-BASED INSPECTOR (3 HRS) OF THE METEOROLOGICAL
        MONITORING SYSTEM.  THE INSPECTOR EXAMINED THE MONITORING
        EQUIPMENT AND THE SUPPORT FACILITIES IN THE EMERGENCY OPERA-
        TIONS FACILITY AND REVIEWED LICENSEE ACTIONS TAKEN IN RESPONSE
        TO THE FEBRUARY 11, 1980 CONFIRMATORY ORDER IN THE FOLLOWING
        AREAS: PRIMARY METEOROLOGICAL MEASUREMENTS PROGRAM; BACKUP
        METEOROLOGICAL MEASUREMENTS PROGRAM; REAL-TIME PREDICTIONS OF
        ATMOSPHERIC EFFLUENT TRANSPORT AND DIFFUSION; AND REMOTE
        INTERROGATION OF THE ATMOSPHERIC MEASUREMENT AND PREDICTION
        SYSTEMS.  NO NONCOMPLIANCES WERE IDENTIFIED.
```

REPORTS RECEIVED FROM LICENSEE

```
==========================================================================
 NUMBER    DATE OF    DATE OF    SUBJECT
            EVENT      REPORT
==========================================================================
 NONE
==========================================================================
```

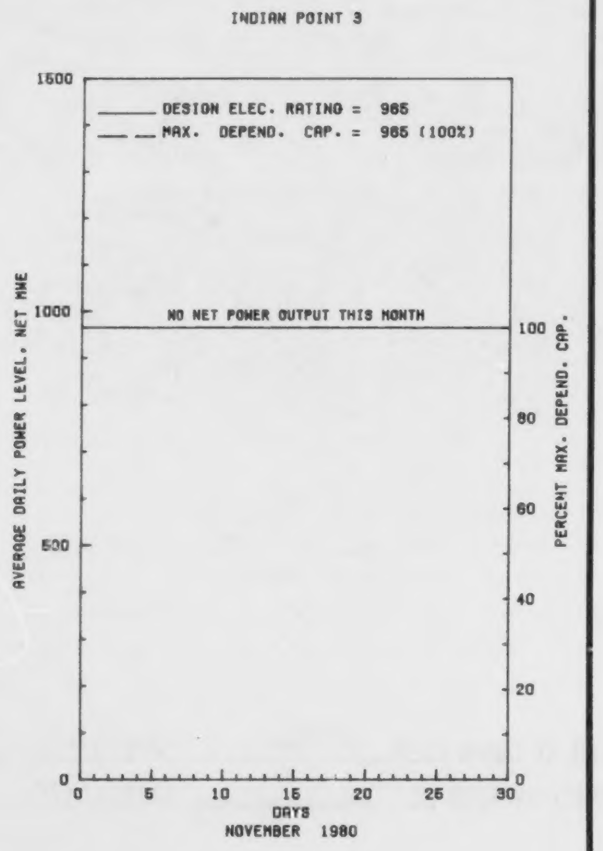
REVIEWED BY OMPA LOEB: P E Bobe **DATE:** 1/16/81

REVIEWED BY NRR DL: L. a. Olsha **DATE:** 1/19/81

best copy available

INDIAN POINT 3

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-286
2. Utility Contact: C. Connell (914) 739-8200 x242
3. Licensed Thermal Power (MWt): 3025
4. Nameplate Rating (Gross MWe): 1126 x 0.9 = 1013
5. Design Electrical Rating (Net MWe): 965
6. Maximum Dependable Capacity (Gross MWe): 1000
7. Maximum Dependable Capacity (Net MWe): 965
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): 917
10. Reasons for Restrictions, If Any:
TURBINE BLADE REMOVAL

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	8,040.0	37,297.0
12. Hours Reactor Was Critical	.0	4,856.3	26,441.2
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	.0	4,385.9	25,418.7
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	10,800,817	66,905,284
17. Gross Electrical Energy (MWH)	0	3,017,310	21,382,411
18. Net Electrical Energy (MWH)	0	2,871,081	20,514,518
19. Unit Service Factor	.0	54.6	68.2
20. Unit Availability Factor	.0	54.6	68.2
21. Unit Capacity Factor (MDC Net)	.0	37.0	57.0
22. Unit Capacity Factor (DER Net)	.0	37.0	57.0
23. Unit Forced Outage Rate	.0	20.1	7.4

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 12/04/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
29	09/30/80	S	168.0	B	4		ZZ	ZZZZZZ	TURBINE OUTAGE (CONTINUED)
30	11/06/80	S	432.0	B	9		AB	XXXXXX	FIRE PROTECTION MODIFICATIONS
31	11/25/80	S	120.0	B	9		SB	HTEXCH	FAN COOLER UNIT, INSPECTION, REPAIR AND TESTING.

SUMMARY

THE UNIT REMAINED SHUTDOWN FOR THE ENTIRE MONTH OF NOVEMBER. THE TURBINE REPAIRS WERE COMPLETED ON NOVEMBER 6, 1980. THE FIRE PROTECTION SYSTEM MODIFICATIONS CONTINUED TO BE INSTALLED. REPAIRS WERE PERFORMED ON VARIOUS LEAKS ON THE COOLING COILS OF THE FAN COOLER UNITS.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

KEWAUNEE

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1. Location (state/county) (nearest population center)	Wisconsin/Kewaunee 27 miles E of Green Bay
2. Electric Reliability Council	Mid-America Interpool Network
3. Type of Reactor	PWR
4. Power Level (MWT)	1650
5. Design Electrical Rating (MWE Net)	535
6. Date of Initial Criticality	March 7, 1974
7. Date of Commercial Operation	June 1974
8. Condenser Cooling Method	Once through condenser and return to Lake Michigan
9. Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10. Licensee	Wisconsin Public Service Corp.
11. Corporate Address	P. O. Box 1200 Green Bay, Wisconsin 54305
12. Corporate Contact	E. R. Mathews Power Supply & Eng.
13. Architect/Engineer	Fluor Power Services Corp.
14. Nuclear Steam Supply System	Westinghouse
15. Constructor	Pioneer Services & Eng. Co.

Regulatory Information

16. IE Region responsible	III
17. IE Branch Chief	R. Heishman
18. IE Section Chief	R. F. Warnick
19. IE Resident Inspector	R. L. Nelson
20. Licensing Project Manager	R. B. Licciardo X28362
21. Docket Number	50-305
22. License Number/Date of Issue	DPR-43 issued 12/21/73
23. Public Document Room	Kewaunee Public Library 822 Juneau Street Kewaunee, Wisconsin 54216

INSPECTION STATUS

ENFORCEMENT STATUS

10 CFR 50, APPENDIX B, CRITERION XVI, STATES IN PART THAT MEASURES SHALL BE ESTABLISHED TO ASSURE THAT CONDITIONS ADVERSE TO QUALITY, SUCH AS FAILURES, MALFUNCTIONS, DEFICIENCIES, DEVIATIONS, DEFECTIVE MATERIAL AND EQUIPMENT, AND NONCONFORMANCES ARE PROMPTLY IDENTIFIED AND CORRECTED. IN THE CASE OF SIGNIFICANT CONDITIONS ADVERSE TO QUALITY, THE MEASURES SHALL ASSURE THAT THE CAUSE OF THE CONDITION IS DETERMINED AND CORRECTIVE ACTION TAKEN TO PRECLUDE REPETITION. OPERATIONAL QUALITY ASSURANCE PROGRAM (OQAP), SECTION 11, "NONCOMPLIANCE AND CORRECTIVE ACTION", REQUIRES THAT PLANT NONCONFORMANCES BE ANALYZED TO DETERMINE IF CONDITIONS ADVERSE TO QUALITY ARE SIGNIFICANT IN NATURE AND THAT MEASURES SHALL BE PROVIDED TO ENSURE THAT CORRECTIVE ACTION IS TAKEN TO PRECLUDE RECURRENCE OF SIGNIFICANT CONDITIONS. SECTION 3.1.8 OF THIS OQAP STATES THAT CONDITIONS OF A RECURRING NATURE SHALL BE CONSIDERED AS SIGNIFICANT CONDITIONS. CONTRARY TO THE ABOVE, THE INSPECTOR FOUND A NUMBER OF CASES WHERE NO ACTION WAS TAKEN TO ANALYZE PLANT NONCONFORMANCES FOR TRENDS AND/OR REPETITION WHEN VENDORS REPETITIVELY SUPPLIED NONCONFORMING MATERIAL TO THE FACILITY. IN ADDITION, NO CORRECTIVE ACTION HAS BEEN TAKEN TO RESOLVE REPETITIVE DEFICIENCIES IDENTIFIED BY WPS QA CONCERNING THE FACT THAT THE KEWAUNEE PLANT DOES NOT HAVE AN IMPLEMENTED PROGRAM TO ESTABLISH SHELF-LIFE CONTROLS.
(8023 INFRACTION)

10 CFR 50, APPENDIX B, CRITERION XVII, STATES IN PART THAT CONSISTENT WITH APPLICABLE REGULATORY REQUIREMENTS, THE APPLICANT SHALL ESTABLISH REQUIREMENTS CONCERNING RECORD RETENTION, SUCH AS DURATION, LOCATION, AND ASSIGNED RESPONSIBILITY. OPERATIONAL QUALITY ASSURANCE PROGRAM DESCRIPTION DATED JANUARY 20, 1978, SECTION 2.3 STATES THAT THE OPERATIONAL QUALITY ASSURANCE PROGRAM (OQAP) MANUAL TRANSLATES THE REQUIREMENTS OF REGULATORY GUIDE 1.88 REVISION 2 INTO A REFERENCE MANUAL FOR THE OQAP. REGULATORY GUIDE 1.88 STATES THAT THE REQUIREMENTS AND RECOMMENDATIONS INCLUDED IN ANSI N45.2.9-1974 ARE ACCEPTABLE TO THE NRC. ANSI N45.2.9-1974 SECTIONS 5.3 AND 6.2 REQUIRES THAT PRIOR TO STORAGE OF RECORDS IN A QUALITY ASSURANCE RECORD FILE, A WRITTEN STORAGE PROCEDURE SHALL BE PREPARED AND A CUSTODIAN SHALL BE DESIGNATED WITH THE RESPONSIBILITY TO ENFORCE THE PROCEDURE AND THAT A LIST SHALL BE GENERATED DESIGNATING THOSE PERSONNEL WHO SHALL HAVE ACCESS TO THE FILES. CONTRARY TO THE ABOVE, THE KEWAUNEE PLANT HAS NOT DESIGNATED A RECORD CUSTODIAN NOR HAS A PERSONNEL ACCESS LIST BEEN GENERATED.
(8023 INFRACTION)

10 CFR 50, APPENDIX B, CRITERION VII, STATES IN PART THAT MEASURES SHALL BE ESTABLISHED TO ASSURE THAT PURCHASED MATERIAL, EQUIPMENT, AND SERVICES, WHETHER PURCHASED DIRECTLY OR THROUGH CONTRACTORS AND SUBCONTRACTORS, CONFORM TO THE PROCUREMENT DOCUMENTS. THESE MEASURES SHALL INCLUDE PROVISIONS, AS APPROPRIATE, FOR SOURCE EVALUATION AND SELECTION, OBJECTIVE EVIDENCE OF QUALITY FURNISHED BY THE CONTRACTOR OR SUBCONTRACTOR, INSPECTION AT THE CONTRACTOR OR SUBCONTRACTOR SOURCE, AND EXAMINATION OF PRODUCTS UPON DELIVERY. DOCUMENTARY EVIDENCE THAT MATERIAL AND EQUIPMENT CONFORM TO THE PROCUREMENT REQUIREMENTS SHALL BE AVAILABLE AT THE NUCLEAR POWER PLANT SITE PRIOR TO INSTALLATION OR USE OF SUCH MATERIAL AND EQUIPMENT. CONTRARY TO THE ABOVE, THE INSPECTOR FOUND THAT WISCONSIN PUBLIC SERVICE CORPORATION HAS NOT ESTABLISHED ADEQUATE MEASURES TO CONTROL PURCHASED MATERIAL AS FOLLOWS: (A) ITEMS WERE RELEASED FOR INSTALLATION AND ARE BEING USED IN QA-1 SYSTEMS PRIOR TO RECEIPT OF DOCUMENTARY EVIDENCE THAT THEY CONFORM TO THE PROCUREMENT DOCUMENTS. IN ADDITION, ITEMS RECEIVED WITH DOCUMENTATION ARE BEING RELEASED FOR USE PRIOR TO A TECHNICAL REVIEW OF THE DOCUMENTS TO ASSURE CONFORMANCE TO PROCUREMENT REQUIREMENTS. AN EXAMPLE OF AN ITEM RELEASED AND USED PRIOR TO RECEIPT OF DOCUMENTATION REQUIRED BY THE PURCHASE ORDER WAS A STARTER FOR THE PLANT DIESEL GENERATOR. ADDITIONAL ITEMS WERE ALSO NOTED: (B) NONCONFORMING ITEMS (NCR'S) WERE NOT REVIEWED AND SIGNED OFF BY THE PLANT PERFORMANCE ENGINEER PRIOR TO RELEASE OF THESE ITEMS AS REQUIRED BY ADMINISTRATIVE DIRECTIVE 3.8.
(8023 INFRACTION)

10 CFR 50, APPENDIX B, CRITERION X, STATES IN PART THAT INSPECTION OF ACTIVITIES AFFECTING QUALITY SHALL BE PERFORMED BY INDIVIDUALS OTHER THAN THOSE WHO PERFORMED THE ACTIVITY BEING INSPECTED. OPERATIONAL QUALITY ASSURANCE PROGRAM DESCRIPTION DATED JANUARY 20, 1973, STATES IN THE INTRODUCTION SECTION THAT THE PROGRAM COMPLIES WITH THE PROVISIONS OF ANSI N18.7-1976, "ADMINISTRATIVE CONTROLS AND QUALITY ASSURANCE FOR THE OPERATIONAL PHASE OF NUCLEAR POWER PLANTS." ANSI N18.7-1976, SECTION 5.2.6 REQUIRES THAT ELECTRICAL JUMPERS AND LIFTED ELECTRICAL LEADS SHALL BE CONTROLLED BY APPROVED PROCEDURES WHICH SHALL INCLUDE A REQUIREMENT FOR INDEPENDENT VERIFICATION. CONTRARY TO THE ABOVE, ACD 8.4 TEMPORARY CHANGES AND CORRECTIVE/REPAIR MAINTENANCE PROCEDURE SP003 DO NOT INCLUDE A REQUIREMENT FOR INDEPENDENT VERIFICATION OF THE INSTALLATION AND REMOVAL OF ELECTRICAL JUMPERS.
(8023 INFRACTION)

TECHNICAL SPECIFICATIONS SECTION 6.8.1 STATES IN PART THAT WRITTEN PROCEDURES SHALL BE IMPLEMENTED THAT MEET THE REQUIREMENTS AND RECOMMENDATIONS OF SECTION 5.1 AND 5.3 OF ANSI N18.7-1972. ANSI N18.7-1972 SECTION 5.3 STATES THAT NUCLEAR POWER PLANTS SHALL BE OPERATED IN ACCORDANCE WITH WRITTEN PROCEDURES. CONTRARY TO THE ABOVE, THE FOLLOWING WRITTEN PROCEDURES WERE NOT IMPLEMENTED: (A) OPERATIONAL QUALITY ASSURANCE PROGRAM, SECTION 10, "CALIBRATION CONTROL," SECTION 3.5.1 REQUIRES THAT MEASUREMENT AND TEST EQUIPMENT SHALL BE SUITABLY MARKED TO INDICATE CALIBRATION DATE, BY WHOM THE DEVICE WAS CALIBRATED, AND THE NEXT SCHEDULED CALIBRATION DATE. TWO DEADWEIGHT TESTERS (CONTROL NUMBERS 91150 AND 91151) WERE RETURNED FROM VENDOR CALIBRATION AND PLACED INTO SERVICE AS REFERENCE STANDARDS WITHOUT PLACING THE APPROPRIATE CALIBRATION STICKERS ON THE INSTRUMENTS. DEADWEIGHT TESTER NO. 91150, BEARING A CALIBRATION STICKER ON BOTH THE INSTRUMENT AND THE WEIGHT BOX SHOWING A CALIBRATION DUE DATE OF JANUARY, 1980, WAS SUBSEQUENTLY USED TO CALIBRATE VARIOUS PIECES OF SHOP AND FIELD TEST AND CALIBRATION EQUIPMENT ON MARCH 14, 17, 18, MAY 1 AND AUGUST 7, 1980; (B) ADMINISTRATIVE CONTROL DIRECTIVE 8.6, "SURVEILLANCE PROCEDURES," SECTION 5.5.2.1.C STATES, IN PART, THAT A SURVEILLANCE PROCEDURE EXCEPTION REPORT SHALL BE INITIATED BY THE PERSON RESPONSIBLE FOR PERFORMING THE SURVEILLANCE TEST...IF A PROBLEM OR OUT OF ACCEPTABLE TOLERANCE CONDITION IS ENCOUNTERED DURING THE SURVEILLANCE TEST. SURVEILLANCE TEST, SP-10-111, "INSPECTION OF DIESEL GENERATOR, REQUIRES THAT BEFORE STARTING THE DIESEL THAT THE ENGINE COOLANT TEMPERATURE BE CHECKED AND THAT THE NORMAL CONDITION BE GREATER THAN 100 DEGREES F. THE TEST RAN ON THE 1A DIESEL ON MAY 21, 1980, INDICATED THE TEMPERATURE WAS 90 DEGREES F. NO SURVEILLANCE PROCEDURE EXCEPTION REPORT WAS INITIATED. OTHER SIMILAR EXAMPLES WERE NOTED TO HAVE OCCURRED; (C) PROCEDURE ICP 33.1 USED TO CALIBRATE SAFETY INJECTION PUMP SUCTION GAUGES PI-926 (11281A) AND PI-927 (11280A) ON DECEMBER 13, 1976, INDICATES THAT THE GAUGES WERE FOUND TO BE OUT OF TOLERANCE. THE GAUGES WERE ADJUSTED AND RETESTED. THE RETEST RESULTS WERE ALSO OUT OF TOLERANCE. THE COMPLETED DATA SHEETS WERE REVIEWED AND ACCEPTED AND NO ACTION WAS TAKEN TO CORRECT THE DISCREPANCY NOR WAS A EXCEPTION REPORT INITIATED.
(8023 INFRACTION)

10 CFR 50, APPENDIX B, CRITERION II, STATES IN PART THAT THE PROGRAM SHALL PROVIDE FOR INDOCTRINATION AND TRAINING OF PERSONNEL PERFORMING ACTIVITIES AFFECTING QUALITY AS NECESSARY TO ASSURE THAT SUITABLE PROFICIENCY IS ACHIEVED AND MAINTAINED. OPERATIONAL QUALITY ASSURANCE PROGRAM, SECTION 13, STATES THAT THE TRAINING PROGRAM SHALL PROVIDE SUITABLE TRAINING OF OTHER STATION PERSONNEL, WHO DO NOT HAVE TO OBTAIN AN NRC OPERATING LICENSE, SUCH AS SUPERVISORS, PROFESSIONALS OPERATORS, TECHNICIANS AND MAINTENANCE PERSONNEL; AND THAT THE TRAINING SHALL BE PROVIDED TO ACHIEVE SPECIAL SKILLS REQUIRED IN THE PERFORMANCE OF PLANT OPERATION, INCLUDINGJOB-RELATED PROCEDURES AND INSTRUCTIONS. CONTRARY TO THE ABOVE, A TRAINING PROGRAM FOR TECHNICIANS AND MAINTENANCE PERSONNEL HAD NOT BEEN ESTABLISHED AS REQUIRED. IN ADDITION, ON-THE-JOB TRAINING RECORDS ARE NOT BEING MAINTAINED CURRENT.
(8023 INFRACTION)

10 CFR 50.59, REQUIRES IN PART THAT A PROPOSED CHANGE TO A FACILITY BE REVIEWED TO DETERMINE THAT IT DOES NOT INVOLVE A UNREVIEWED SAFETY QUESTION; 10 CFR 50, APPENDIX B, CRITERION III REQUIRES IN PART THAT DESIGN CHANGES, INCLUDING FIELD CHANGES BE SUBJECT TO DESIGN CONTROL MEASURES COMMENSURATE WITH THOSE APPLIED TO THE ORIGINAL DESIGN. TECHNICAL SPECIFICATION 6.8.1 REQUIRES THAT THE LICENSEE IMPLEMENT ADMINISTRATIVE POLICIES AND PROCEDURES. CONTRARY TO THE ABOVE, CHANGES WERE MADE TO THE CONTROL ROOM AIR CONDITIONER SERVICE WATER STRAINER DRAIN, A QA TYPE I AND SEISMIC CLASS 1 SYSTEM, ON OCTOBER 30, 1979 WITHOUT REVIEWS OR CONTROLS AND WITHOUT IMPLEMENTING ADMINISTRATIVE CONTROL DIRECTIVES 8.5, 8.4, OR 8.9 WHICH ARE THE ADMINISTRATIVE PROCEDURES THE LICENSEE HAS APPROVED TO COMPLY WITH THE APPLICABLE REGULATORY REQUIREMENTS.
(8023 INFRACTION)

10 CFR 50, APPENDIX B, CRITERIA IV, STATES IN PART THAT MEASURES SHALL BE ESTABLISHED TO ASSURE THAT APPLICABLE REGULATORY REQUIREMENTS, DESIGN BASES, AND OTHER REQUIREMENTS WHICH ARE NECESSARY TO ASSURE ADEQUATE QUALITY ARE SUITABLY REFERENCED IN THE DOCUMENTS FOR PROCUREMENT OF MATERIAL, EQUIPMENT AND SERVICES, WHETHER PURCHASED BY THE APPLICANT OR BY ITS CONTRACTORS OR SUBCONTRACTORS. OPERATING QUALITY ASSURANCE PROGRAM SECTION 6, "PROCUREMENT CONTROL", STATES THAT THE DIRECTIVES SHALL PROVIDE FOR INCLUSION OR REFERENCING OF THE FOLLOWING INFORMATION AS APPROPRIATE: QUALITY ASSURANCE PROGRAM REQUIREMENTS; RIGHT OF ACCESS; AND 10 CFR 21 REQUIREMENTS. CONTRARY TO THE ABOVE, MEASURES HAVE NOT BEEN ADEQUATELY IMPLEMENTED FOR THE PLACEMENT OF PURCHASE ORDERS. TEN OUT OF TWENTY PURCHASE ORDER DOCUMENTS, SPECIFIED NEITHER QUALITY ASSURANCE PROGRAM REQUIREMENTS REFERENCES TO 10 CFR 50, APPENDIX B, NOR 10 CFR 21 PROVISIONS. OF THE TWENTY PURCHASE ORDERS REVIEWED, ALL TWENTY FAILED TO REFERENCE A RIGHT OF ACCESS TO THE SUPPLIERS FACILITIES. THE PURCHASE ORDERS REVIEWED WERE FOR QA-1, SAFETY RELATED ITEMS.
(8023 INFRACTION)

REPORTS RECEIVED FROM LICENSEE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

+ ROUTINE PLANT OPERATION.

\+LAST IE SITE INSPECTION DATE: AUGUST 18-22, 25-29; SEPTEMBER 4-5 AND 29, 1980

+ INSPECTION REPORT NO: 80-23

SUMMARY

INSPECTION SUMMARY

+ INSPECTION ON SEPTEMBER 1-5, 8-12, 15-19, 22-26, AND 29-30, (80-20): ROUTINE RESIDENT INSPECTION OF OPERATIONAL SAFETY VERIFICATION, MONTHLY MAINTENANCE OBSERVATION, MONTHLY SURVEILLANCE OBSERVATION, LICENSEE EVENT REPORTS FOLLOWUP, IE BULLETIN FOLLOWUP, IE CIRCULAR FOLLOWUP, FOLLOWUP ON HEADQUARTERS REQUESTS, FOLLOWUP ON REGIONAL REQUESTS, AND INDEPENDENT INSPECTION. THE INSPECTION INVOLVED 192 INSPECTOR-HOURS ONSITE BY TWO NRC INSPECTORS INCLUDING 28 INSPECTOR-HOURS ONSITE DURING OFF-SHIFTS. NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

+ INSPECTION ON AUGUST 18-22, 25-29; SEPTEMBER 4-5 AND 29, (80-23): ANNOUNCED ROUTINE INSPECTION OF THE LICENSEE'S IMPLEMENTATION OF THE QUALITY ASSURANCE PROGRAM IN THE FOLLOWING AREAS: QA/QC ADMINISTRATION; QUALIFICATION OF PERSONNEL; DESIGN CHANGES AND MODIFICATIONS; RECORDS CONTROL; RECEIPT, STORAGE AND HANDLING; TESTS AND EXPERIMENTS; PROCUREMENT CONTROL; DOCUMENT CONTROL; OFFSITE REVIEW COMMITTEE; AUDITS; TEST AND MEASUREMENT EQUIPMENT; SURVEILLANCE AND CALIBRATION; MAINTENANCE; OFFSITE SUPPORT STAFF; AND CORRECTIVE ACTION. THE INSPECTION INVOLVED 256 INSPECTOR-HOURS, 224 OF THESE HOURS WERE ONSITE BY FOUR NRC INSPECTORS, AND 32 HOURS WERE AT THE LICENSEE'S CORPORATE OFFICE BY TWO NRC INSPECTORS. NONE OF THE INSPECTOR-HOURS WERE SPENT ONSITE DURING OFF-SHIFTS. OF THE 15 AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN 7 AREAS, 8 APPARENT ITEMS OF NONCOMPLIANCE WERE IDENTIFIED IN 8 AREAS (FAILURE TO FOLLOW PROCEDURES RELATING TO CALIBRATION AND SURVEILLANCE; FAILURE TO TAKE CORRECTIVE ACTION; FAILURE TO INCLUDE REQUIREMENTS IN PROCUREMENT DOCUMENTS; FAILURE TO ESTABLISH RECORD CONTROL ACCESSIBILITY; FAILURE TO IMPLEMENT DESIGN CHANGE CONTROL; FAILURE TO ESTABLISH TRAINING PROGRAM FOR TECHNICIANS AND MAINTENANCE PERSONNEL; FAILURE TO HAVE ADEQUATE PROCEDURE PERTAINING TO INSTALLATION AND REMOVAL OF ELECTRICAL JUMPERS; AND FAILURE TO CONTROL PURCHASED MATERIAL).

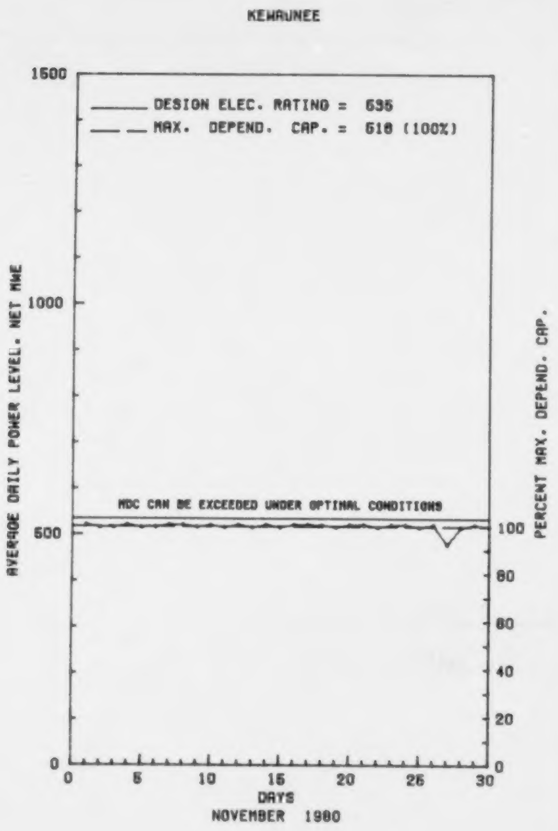
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-31/ 03L-0	08/22/80	09/19/80	CST LEVEL BELOW TS LIMIT.
80-32/ 03L-0	09/25/80	10/24/80	VALVE PLUG FROM BYPASS VALVE DISLODGED AND RESTRICTED FLOW IN HOT LOOP OF RCS BYPASS.
80-33/ 03L-0	10/01/80	10/24/80	CST LEVEL WENT BELOW TS.
80-34/ 03L-0	10/14/80	11/12/80	WELD FAILURE - FAULTY FAN BLADE ASSEMBLY.

REVIEWED BY OMPA LOEB: P E Bole **DATE:** 1/16/81

REVIEWED BY NRR DL: Don Neighbors **DATE:** 1/19/81

KEWAUNEE

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-305
2. Utility Contact: G.RUITER (414) 388-2560 X207
3. Licensed Thermal Power (MWt): 1650
4. Nameplate Rating (Gross MWe): 622 x 0.9 = 560
5. Design Electrical Rating (Net MWe): 535
6. Maximum Dependable Capacity (Gross MWe): 545
7. Maximum Dependable Capacity (Net MWe): 518
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

ITEM 21 CALCULATED BY USING A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	8,040.0	56,641.0
12. Hours Reactor Was Critical	720.0	6,582.7	47,585.2
13. Reactor Reserve Shutdown Hours	.0	152.7	2,330.5
14. Hours Generator On-Line	720.0	6,481.9	46,477.9
15. Unit Reserve Shutdown Hours	.0	.0	10.0
16. Gross Thermal Energy (MWH)	1,183,962	10,347,647	71,498,770
17. Gross Electrical Energy (MWH)	390,700	3,418,500	23,602,300
18. Net Electrical Energy (MWH)	372,522	3,257,714	22,456,722
19. Unit Service Factor	100.0	80.6	82.1
20. Unit Availability Factor	100.0	80.6	82.1
21. Unit Capacity Factor (MDC Net)	99.9	77.6	76.7
22. Unit Capacity Factor (DER Net)	96.7	75.7	74.1
23. Unit Forced Outage Rate	.0	6.7	5.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
REFUELING SHUTDOWN FOR APRIL 18, 1981; 6-WEEK DURATION.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	NONE								

SUMMARY

KEWAUNEE OPERATED AT FULL POWER DURING NOVEMBER, WITH NO OUTAGES OR REDUCTIONS REPORTED.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram.
3--Automatic Scram.
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

LACROSSE

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Wisconsin/Vernon 19 miles S of LaCrosse
2.	Electric Reliability Council	Mid-Continent Area Reliability Coordination Agreement
3.	Type of Reactor	BWR
4.	Power Level (MWT)	165
5.	Design Electrical Rating (MWE Net)	50
6.	Date of Initial Criticality	July 11, 1967
7.	Date of Commercial Operation	November 1, 1969
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Mississippi River

Utility & Contractor Information

10.	Licensee	Dairyland Power Cooperative
11.	Corporate Address	2615 East Avenue South La Crosse, Wisconsin
12.	Corporate Contact	F. Linder
13.	Architect/Engineer	Sargent & Lundy
14.	Nuclear Steam Supply System	Allis-Chalmers
15.	Constructor	Maxson Construction

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. F. HEISHMAN
18.	IE Section Chief	D. C. BOYD
19.	IE Resident Inspector	W. Forney
20.	Licensing Project Manager	J. J. Shea
21.	Docket Number	50-409
22.	License Number/Date of Issue	DPR-45 issued 8/28/73
23.	Public Document Room	LaCrosse Public Library 800 Main Street LaCrosse, Wisconsin 54601

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

FACILITY ITEMS

- LICENSEE PERFORMING FUEL ELEMENT STORAGE WELL FUEL RACK MODIFICATIONS. (80-11)

PLANT STATUS

- IN REFUELING OUTAGE. RESTART SCHEDULED FOR DECEMBER 25, 1980.

+LAST IE SITE INSPECTION DATE: OCTOBER 9-10, 1980

- INSPECTION REPORT NO: 80-12

INSPECTION SUMMARY

- INSPECTION ON SEPTEMBER 1-30, (80-11): ROUTINE, UNANNOUNCED INSPECTION OF THE LICENSEE'S OPERATIONAL SAFETY; SURVEILLANCE; MAINTENANCE; FOLLOWUP ACTION ON IE BULLETINS; LICENSEE EVENT REPORTS; OPEN INSPECTION ITEMS; AND PLANT TRIPS. THE INSPECTION INVOLVED 270 INSPECTOR-HOURS ONSITE BY 4 NRC INSPECTORS INCLUDING 33 INSPECTOR-HOURS DURING OFF-SHIFTS. OF THE SEVEN AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE FOUND.

- INSPECTION ON OCTOBER 9-10, (80-12): INCLUDED A REVIEW OF THE PROTECTED AREA PHYSICAL BARRIERS; DETECTION AIDS; TESTING AND MAINTENANCE; SECURITY PLAN; POWER SUPPLY; AND FOLLOW-UP ON PREVIOUS ITEMS OF NONCOMPLIANCE. THE INSPECTION INVOLVED 11 INSPECTOR-HOURS ONSITE BY ONE NRC INSPECTOR. THE INSPECTION BEGAN DURING THE DAY SHIFT; NO INSPECTION HOURS WERE ACCOMPLISHED DURING OFF-SHIFT PERIODS. THE LICNESEE WAS FOUND TO BE IN COMPLIANCE WITH NRC REQUIREMENTS WITHIN THE AREAS EXAMINED.

REPORTS RECEIVED FROM LICENSEE

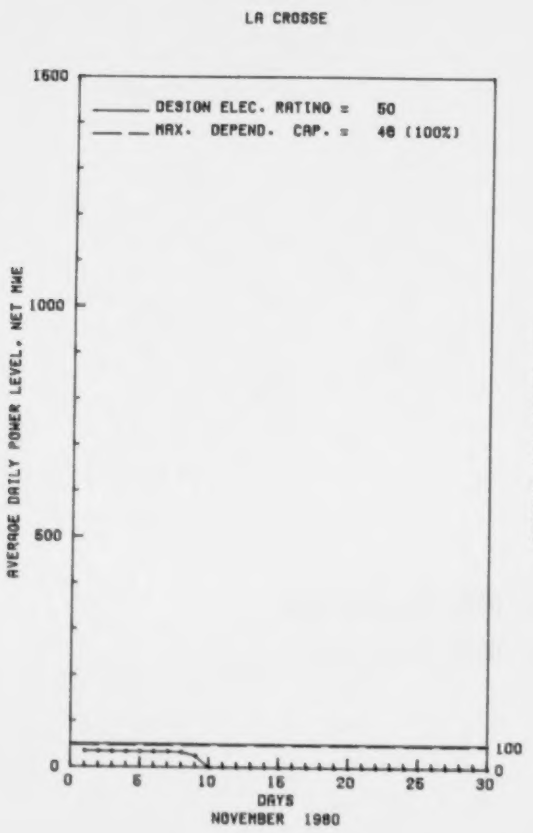
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-11/01T-0	11/10/80	11/24/80	DURING PLANT COOLDOWN, COOLDOWN RATE EXCEEDED TS LIMIT OF 150 DEGREES PER HOUR AT 5 POINTS ON REACTOR VESSEL.

REVIEWED BY OMPA LOEB: DATE: 1-16-81

REVIEWED BY NRR DL: DATE: 1-19-81

LACROSSE

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-409
2. Utility Contact: L. S. Goodman (608) 689-2331
3. Licensed Thermal Power (MWt): 165
4. Nameplate Rating (Gross MWe): 76.8 x 0.85 =65
5. Design Electrical Rating (Net MWe): 50
6. Maximum Dependable Capacity (Gross MWe): 50
7. Maximum Dependable Capacity (Net MWe): 48
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	8,040.0	97,155.0
12. Hours Reactor Was Critical	213.3	6,290.9	64,097.0
13. Reactor Reserve Shutdown Hours	.0	.0	478.0
14. Hours Generator On-Line	213.4	6,026.5	59,031.5
15. Unit Reserve Shutdown Hours	.0	.0	79.0
16. Gross Thermal Energy (MWH)	26,569	799,486	8,084,265
17. Gross Electrical Energy (MWH)	7,683	231,503	2,430,635
18. Net Electrical Energy (MWH)	6,837	215,042	2,247,777
19. Unit Service Factor	29.6	75.0	60.8
20. Unit Availability Factor	29.6	75.0	60.8
21. Unit Capacity Factor (MDC Net)	19.8	55.7	48.2
22. Unit Capacity Factor (DER Net)	19.0	53.5	46.3
23. Unit Forced Outage Rate	.0	13.0	7.1

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): OPERATOR LICENSING EXAMINATIONS, JANUARY 12, 1981, 2 DAYS
25. If Shut Down At End of Report Period, Estimated Date of Startup: 12/26/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-12	11/09/80	S	506.6	C	2				REFUELING OUTAGE COMMENCED.

SUMMARY

LACROSSE REMAINED ON-LINE THRU NOVEMBER 9TH, AND COMMENCED INTO A REFUELING OUTAGE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for License Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

MAINE YANKEE

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Maine/Lincoln 10 miles N of Bath (near Wiscasset)
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2630
5.	Design Electrical Rating (MWE Net)	825
6.	Date of Initial Criticality	October 23, 1972
7.	Date of Commercial Operation	December 28, 1972
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Back River (Atlantic)

Utility & Contractor Information

10.	Licensee	Maine Yankee Atomic Power Co.
11.	Corporate Address	1671 Worcester Road Framingham, Massachusetts 01581
12.	Corporate Contact	D. W. Edwards, Assistant to Vice President
13.	Architect/Engineer	Stone & Webster
14.	Nuclear Steam Supply System	Combustion Engineering
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	IE Section Chief	T. T. Martin
19.	IE Resident Inspector	W. J. LAZARUS (207) 882-7519
20.	Licensing Project Manager	G. Requa (X28478)
21.	Docket Number	50-309
22.	License Number/Date of Issue	DPR-36, 9/15/72
23.	Public Document Room	Wiscasset Public Library High Street Wiscasset, Maine 04578

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

```
      I N S P E C T I O N   S T A T U S

      OTHER ITEMS OF INTEREST

      SYSTEM AND COMPONENT PROBLEMS:

  +   ACCELERATED CORROSION WAS DISCOVERED
      IN THE MAIN CONDENSER WATER BOXES.
      MAGNESIUM ANODES WERE INSTALLED TO
      REPLACE THE DEPLETED ZINC ANODES.

      PLANT STATUS:

  +   THE PLANT OPERATED AT FULL LOAD EXCEPT
      FOR SHORT LOAD REDUCTIONS TO ACCOMPLISH
      CONDENSER REPAIRS.

  + LAST IE SITE INSPECTION DATE:  11/3-29/80

  +         INSPECTION REPORT NO:  50-309/80-18

      SUMMARY

      INSPECTION SUMMARY

  +   50-309/80-14 - SEP 2 - OCT 3:  ROUTINE, REGULAR AND BACKSHIFT
      INSPECTION BY TWO RESIDENT INSPECTORS AND ONE REGIONAL IN-
      SPECTOR (142 HOURS).  AREAS INSPECTED INCLUDED THE CONTROL
      ROOM, TURBINE BUILDING, PRIMARY AUXILIARY BUILDING, SPRAY
      BUILDING AND AUXILIARY FEED PUMP ROOM.  ACTIVITIES/RECORDS
      INSPECTED INCLUDED RADIATION PROTECTION, PHYSICAL SECURITY,
      PLANT OPERATIONS, MAINTENANCE AND SURVEILLANCE TESTING, ON-
      SITE VERIFICATION OF IEB 79-01B, FOLLOWUP ON PREVIOUS INSPECTION
      FINDINGS, FOLLOWUP ON SELECTED IE BULLETINS, FOLLOWUP ON IE:HQTRS
      REQUESTS AND FOLLOWUP ON LICENSEE PREPARATION FOR A STRIKE. NO
      NONCOMPLIANCES WERE IDENTIFIED.

  +   50-309/80-15 - OCT 16:  MANAGEMENT MEETING HELD AT NRC'S
      REQUEST TO DISCUSS THE REGULATORY PERFORMANCE OF THE LICENSEE
      PERFORMANCE (SALP) PROGRAM.  A SUMMARY OF THE LICENSEE
      EVALUATION PERFORMANCE WAS PRESENTED.  AREAS OF CONCERN WERE
      DISCUSSED WITH CORPORATE MANAGEMENT.  PERFORMANCE OF THE
      LICENSEE WAS CONSIDERED TO BE ACCEPTABLE.
```

REPORTS RECEIVED FROM LICENSEE

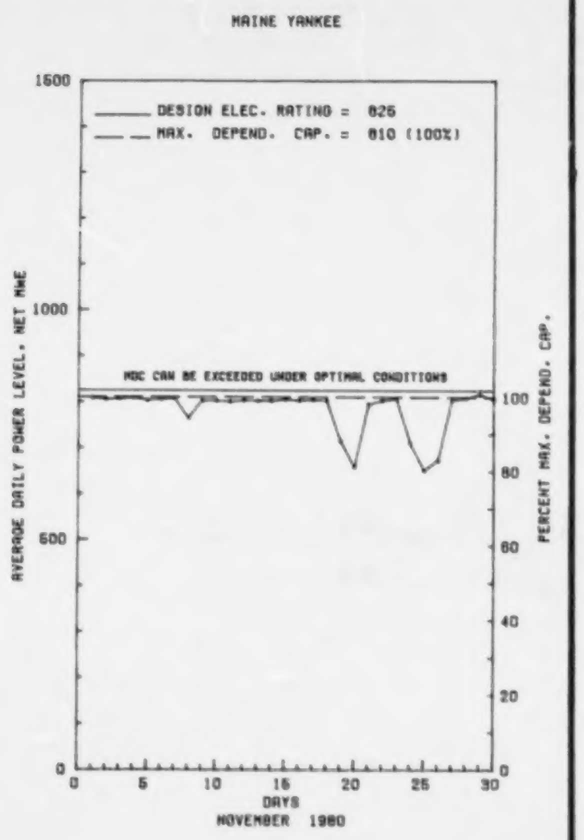
```
=====================================================================
 NUMBER    DATE OF    DATE OF    SUBJECT
            EVENT      REPORT
=====================================================================
 NONE
=====================================================================
```

REVIEWED BY OMPA LOEB: MR Beebe **DATE:** 01/16/81

REVIEWED BY NRR DL: G. Requa **DATE:** 01/19/81

MAINE YANKEE

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-309
2. Utility Contact: F.C. BEERS (617) 366-9011
3. Licensed Thermal Power (MWt): 2630
4. Nameplate Rating (Gross MWe): 900 x 0.9 = 810
5. Design Electrical Rating (Net MWe): 825
6. Maximum Dependable Capacity (Gross MWe): 850
7. Maximum Dependable Capacity (Net MWe): 810
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

CAPACITY FACTORS CALCULATED WITH A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	8,040.0	70,668.6
12. Hours Reactor Was Critical	720.0	6,133.3	57,343.2
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	720.0	5,934.1	55,451.8
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,796,502	13,196,805	118,638,808
17. Gross Electrical Energy (MWH)	590,580	4,319,030	38,892,210
18. Net Electrical Energy (MWH)	563,245	4,108,989	36,936,175
19. Unit Service Factor	100.0	73.8	78.5
20. Unit Availability Factor	100.0	73.8	78.5
*21. Unit Capacity Factor (MDC Net)	96.6	63.1	67.6
22. Unit Capacity Factor (DER Net)	94.8	61.9	66.4
23. Unit Forced Outage Rate	.0	0.0	7.0

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	11/08/80	S	24.0	B	5		HA	VALVEX	LOAD REDUCTION TO 79%. ROUTINE TURBINE VALVE SURVEILLANCE TESTING.
	11/19/80	F	49.3	A	5		HC	HTEXCH	LOAD REDUCTION TO 80%. CONDENSER TUBE LEAK IN THE "C" WATERBOX 1 TUBE PLUGGED.
	11/24/80	F	63.2	A	5		HC	HTEXCH	LOAD REDUCTION TO 79%. CLEAN DEBRIS OFF CONDENSER TUBESHEETS.
	11/30/80	F	4.0	A	5		HC	HTEXCH	LOAD REDUCTION TO 80%. SANDBLASTING OF THE MAIN CONDENSER TUBESHEETS (GALVANIC CORROSION).

SUMMARY

THE UNIT WAS ON LINE THE ENTIRE MONTH WITH SEVERAL LOAD REDUCTIONS. ON NOVEMBER 19, A LEAKING CONDENSER TUBE WAS LOCATED AND PLUGGED. SEVERAL AREAS OF THE TUBESHEET SHOWED INDICATIONS OF GALVANIC CORROSION ATTACK POSSIBLY COUPLED WITH FLUID ERROSION.

LEGEND

[1] Type:
F–Forced
S–Scheduled

[2] Reason:
A–Equipment Failure
B–Maintenance or Test
C–Refueling
D–Regulatory Restriction
E–Operator Training & License Examination
F–Administrative
G–Operational Error
H–Other

[3] Method:
1–Manual
2–Manual Scram.
3–Automatic Scram.
4–Continuations
5–Load Reduction
9–Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

MILLSTONE 1

FACILITY DATA

Facility Description

1. Location (state/county) (nearest population center)	Connecticut/New London 5 miles SW of New London
2. Electric Reliability Council	Northeast Power Coordinating Council
3. Type of Reactor	BWR
4. Power Level (MWT)	2011
5. Design Electrical Rating (MWE Net)	660
6. Date of Initial Criticality	October 26, 1970
7. Date of Commercial Operation	March 1971
8. Condenser Cooling Method	Once Through
9. Condenser Cooling Water Source	Long Island Sound

Utility & Contractor Information

10. Licensee	Northeast Nuclear Energy Company
11. Corporate Address	P. O. Box 270 Hartford, Connecticut
12. Corporate Contact	D. Switzer, President
13. Architect/Engineer	Ebasco
14. Nuclear Steam Supply System	GE
15. Constructor	Ebasco

Regulatory Information

16. IE Region responsible	I
17. IE Branch Chief	E. Brunner
18. IE Section Chief	R. KEIMIG
19. IE Resident Inspector	J. SHEDLOSKY (203-447-3179) FTS: 642-7287
20. Licensing Project Manager	J. J. Shea
21. Docket Number	50-245
22. License Number/Date of Issue	DPR-21, 10/7/70
23. Public Document Room	Waterford Public Library Rope Ferry Road Route 156 Waterford, Connecticut 06385

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

```
INSPECTION STATUS

OTHER ITEMS OF INTEREST

+    SYSTEMS AND COMPONENT PROBLEMS:

+    THE LICENSEE IS EVALUATING NDT INDICATIONS
     OF DEFECTS IN CLASS I AND CLASS II PORTIONS OF
     THE ISOLATION CONDENSER STEAM SUPPLY PIPING.

     PLANT STATUS:

+    THE PLANT HAS BEEN SHUTDOWN SINCE OCTOBER
     4 FOR A REFUELING OUTAGE.

+    LAST IE SITE INSPECTION DATE:  11/11-14/80

+         INSPECTION REPORT NO:  50-245/80-23

     SUMMARY

     INSPECTION SUMMARY

+    50-245/80-16 - SEP 21-26:  ROUTINE, UNANNOUNCED, ONSITE
     PHYSICAL SECURITY INSPECTION BY TWO REGION-BASED INSPECTORS
     (72 HOURS) OF:  SECURITY PLAN; SECURITY ORGANIZATION; PHYSICAL
     BARRIERS (PROTECTED AND VITAL AREAS); LIGHTING; ACCESS CONTROL
     (IDENTIFICATION, AUTHORIZATION AND BADGING); ACCESS CONTROLS
     (SEARCH AND ESCORTING); ACCESS CONTROLS (VITAL AREAS); ALARM
     STATIONS; DETECTION AIDS; ASSESSEMENT AIDS; COMMUNICATIONS;
     TESTING AND MAINTENANCE; COMPENSATORY MEASURES; POWER SUPPLY;
     RESPONSE; LOCKS, KEYS AND COMBINATIONS; AND RECORDS AND RE-
     PORTS.  ALSO, A FOLLOWUP TO PREVIOUSLY IDENTIFIED ITEMS OF
     NONCOMPLIANCE AND A FOLLOWUP TO IMMEDIATE ACTION LETTER (IAL
     80-28).  NO NONCOMPLIANCES WERE IDENTIFIED.

+    50-245/80-19 - OCT 6-9:  ROUTINE, UNANNOUNCED ONSITE IN-
     SPECTION BY THREE REGION-BASED INSPECTORS (50 HOURS) OF THE
     CONTAINMENT INTEGRATED LEAK RATE TEST PROCEDURE; CONTAINMENT
     LOCAL LEAK RATE TESTING; SURVEILLANCE OF PIPE SUPPORT AND RE-
     STRAINT SYSTEMS; AND, LICENSEE ACTION ON PREVIOUS INSPECTION
     FINDINGS.  NO NONCOMPLIANCES WERE IDENTIFIED.
```

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

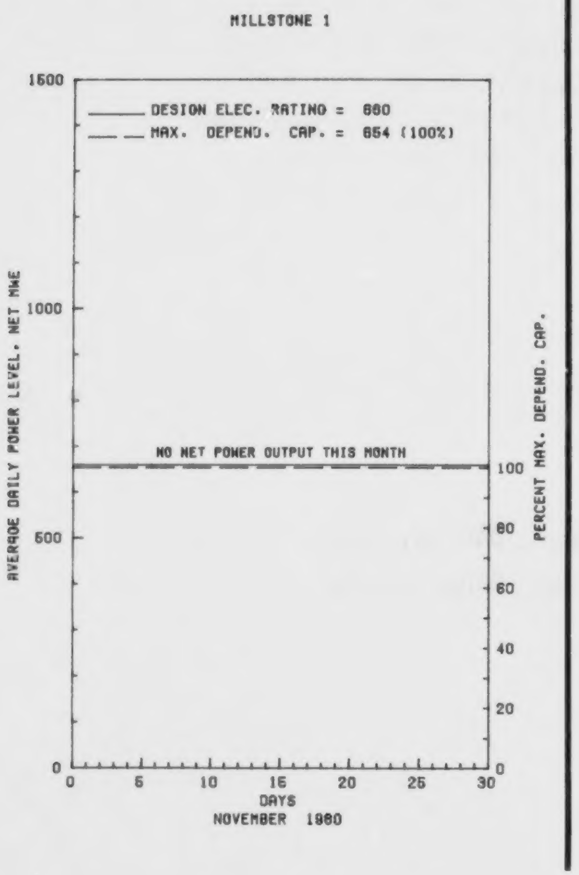
```
==============================================================================
NUMBER     DATE OF    DATE OF    SUBJECT
            EVENT      REPORT
==============================================================================
80-002-E  09/22/80   10/21/80   FAILURE TO PERFORM WEEKLY SURVEILLANCE TESTING OF OFF-GAS HYDROGEN
                                 ANALYZERS WITHIN THE REQUIRED TIME INTERVAL
80-015/    10/07/80   10/24/80  SETPOINT DRIFT-REACTOR VESSEL LOW WATER LEVEL RPS AND CONTAINMENT
 03L                             ISOLATION INSTRUMENT
80-016/   10/23/80   11/06/80   DEGRADATION OF BWR JET PUMP ASSEMBLY BEAMS.  IGSC CRACKING OF ONE
 01T                             BEAM CAUSING REJECTION AND FOUR BEAMS IN MARGINAL CONDITION
80-018/   11/05/80   11/19/80   DISCOVERY OF FAILED WELDS BETWEEN MAIN STEAM LINE SUPPORTS AND THE
 01T                             DRYWELL SUPPORT STRUCTURE
80-019/   11/06/80   11/20/80   DISCOVERY OF CIRCUMFERENTIAL CRACKING IN ISOLATION CONDENSER STEAM
 01T                             SUPPLY PIPING
==============================================================================
```

REVIEWED BY OMPA LOEB: John J. **DATE:** 1-16-31

REVIEWED BY NRR DL: J. J. Shea **DATE:** 1-19-81

MILLSTONE 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-245
2. Utility Contact: G. HARRAN (203) 447-1792 X655
3. Licensed Thermal Power (MWt): 2011
4. Nameplate Rating (Gross MWe): 735 x 0.9 = 662
5. Design Electrical Rating (Net MWe): 660
6. Maximum Dependable Capacity (Gross MWe): 684
7. Maximum Dependable Capacity (Net MWe): 654
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	8,040.0	87,720.0
12. Hours Reactor Was Critical	.0	6,074.3	66,659.3
13. Reactor Reserve Shutdown Hours	.0	592.3	1,527.3
14. Hours Generator On-Line	.0	6,030.3	64,126.2
15. Unit Reserve Shutdown Hours	.0	26.5	26.5
16. Gross Thermal Energy (MWH)	0	10,382,182	114,430,127
17. Gross Electrical Energy (MWH)	0	3,564,500	38,795,896
18. Net Electrical Energy (MWH)	-2,110	3,392,539	37,032,321
19. Unit Service Factor	.0	75.0	73.1
20. Unit Availability Factor	.0	75.3	73.1
21. Unit Capacity Factor (MDC Net)	.0	64.5	64.6
22. Unit Capacity Factor (DER Net)	.0	63.9	64.0
23. Unit Forced Outage Rate	.0	.2	15.3

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 01/12/81

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
10A	10/04/80	S	720.0	C	4	80-18/1P	CC	SUPORT	THE UNIT WAS SHUTDOWN FOR ANNUAL REFUEL OUTAGE DURING THE ENTIRE MONTH. TWO LER'S RESULTED AND WERE RELATED TO: 1. MAIN STEAM SUPPORT WELDS. 2. ISOLATION CONDENSER NOZZLE WELD. THEIR LER #'S ARE 80-18/1P - 80-19/1T

SUMMARY

THE UNIT CONTINUED REFUELING DURING THE MONTH OF NOVEMBER.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

MILLSTONE 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Connecticut/New London 5 miles SW of New London
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2700
5.	Design Electrical Rating (MWE Net)	870
6.	Date of Initial Criticality	October 17, 1975
7.	Date of Commercial Operation	December 26, 1975
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Long Island Sound

Utility & Contractor Information

10.	Licensee	Northeast Nuclear Energy Company
11.	Corporate Address	P. O. Box 270 Hartford, Connecticut
12.	Corporate Contact	D. Switzer, President
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Combustion Engineering
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	IE Section Chief	R. Keimig
19.	IE Resident Inspector	J. T. SHEDLOSKY (203-447-3179) R. Zimmerman FTS: 642-7287
20.	Licensing Project Manager	E. L. Conner, Jr.
21.	Docket Number	50-336
22.	License Number/Date of Issue	DPR-65, 8/1/75
23.	Public Document Room	Waterford Public Library Rope Ferry Road Route 156 Waterford, Connecticut 06385

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS:

+ THE PLANT HAS BEEN OPERATING AT FULL POWER.

+ LAST IE SITE INSPECTION DATE: 11/4-7/80

+ INSPECTION REPORT NO: 50-336/80-22

SUMMARY

INSPECTION SUMMARY

+ 50-336/80-17 - SEP 21-26: ROUTINE, UNANNOUNCED, ONSITE PHYSICAL SECURITY INSPECTION BY TWO REGION-BASED INSPECTORS (72 HRS) OF: SECURITY PLAN; SECURITY ORGANIZATION; PHYSICAL BARRIERS (PROTECTED AND VITAL AREAS); LIGHTING; ACCESS CONTROL (IDENTIFICATION, AUTHORIZATION AND BADGING); ACCESS CONTROLS (SEARCH AND ESCORTING); ACCESS CONTROLS (VITAL AREAS); ALARM STATIONS; DETECTION AIDS; ASSESSMENT AIDS; COMMUNICATIONS; TESTING AND MAINTENANCE; COMPENSATORY MEASURES; POWER SUPPLY; RESPONSE; LOCKS, KEYS AND COMBINATIONS; AND RECORDS AND REPORTS. ALSO, A FOLLOWUP TO PREVIOUSLY IDENTIFIED ITEMS OF NONCOMPLIANCE AND A FOLLOWUP TO IMMEDIATE ACTION LETTER (IAL 80-28). NO NONCOMPLIANCES WERE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

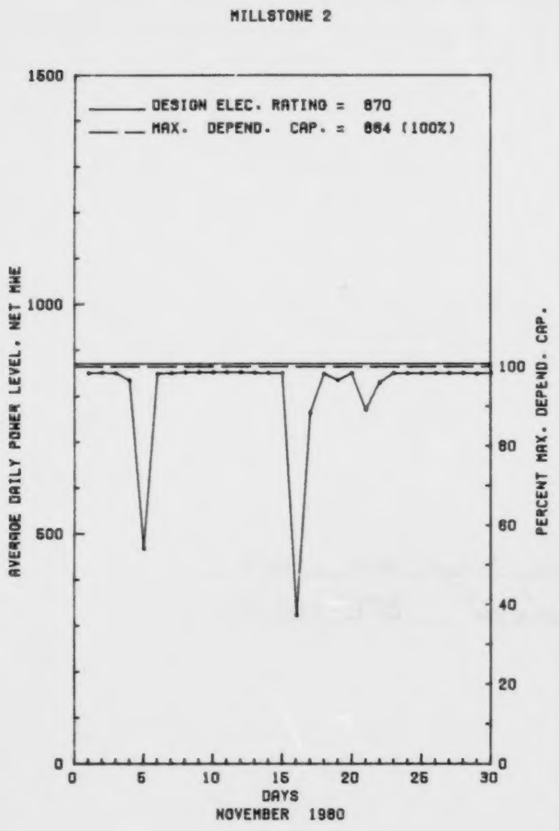
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-032/03L	09/17/80	10/16/80	REACTOR BUILDING CLOSED COOLING WATER SYSTEM CONTAINMENT ISOLATION VALVE FAILURE TO OPERATE
80-033/03L	09/30/80	10/28/80	ONE OF TWO WIDE RANGE FLUX NUCLEAR INSTRUMENTS FAILED DURING A PLANT SHUTDOWN
80-034/03L	10/17/80	11/14/80	TURBINE DRIVEN AUXILIARY FEEDWATER PUMP REMOVED FROM SERVICE TO ALLOW THE REPAIR OF A LEAKING DRAIN VALVE
80-035/03L	10/16/80	11/14/80	FIRE BARRIER BETWEEN CABLE SPREAD ROOM AND EAST ELECTRICAL PENETRATION ROOM BREACHED FOR THE REMOVAL OF TEMPORARY CABLES
80-036/03L	10/23/80	11/20/80	INOPERABLE CHANNEL D RPS TRIP FUNCTIONS OF HIGH POWER, LOCAL POWER DENSITY AND THERMAL MARGIN LOW PRESSURE DUE TO UPPER AND LOWER DETECTION CABLES BEING REVERSED
80-037/03L	10/25/80	11/24/80	SETPOINT DRIFT-RPS CHANNEL D. 15 PERCENT POWER LOCAL

REVIEWED BY OMPA LOEB: MR Beebe **DATE:** 01/16/81

REVIEWED BY NRR DL: E L Conner **DATE:** 1/19/81

MILLSTONE 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

CAPACITY FACTORS CALCULATED WITH A WEIGHTED AVERAGE

1. Docket Number: 50-336
2. Utility Contact: G. H. Houlett (203) 447-1791 x364
3. Licensed Thermal Power (MWt): 2700
4. Nameplate Rating (Gross MWe): 1011 x 0.9 = 910
5. Design Electrical Rating (Net MWe): 870
6. Maximum Dependable Capacity (Gross MWe): 895
7. Maximum Dependable Capacity (Net MWe): 864
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	8,040.0	43,248.0
12. Hours Reactor Was Critical	720.0	5,426.2	30,504.9
13. Reactor Reserve Shutdown Hours	.0	4.5	2,076.9
14. Hours Generator On-Line	720.0	5,204.8	29,025.0
15. Unit Reserve Shutdown Hours	.0	132.8	468.2
16. Gross Thermal Energy (MWH)	1,862,268	13,541,545	71,240,173
17. Gross Electrical Energy (MWH)	607,400	4,437,072	23,021,717
18. Net Electrical Energy (MWH)	585,141	4,253,675	22,033,933
19. Unit Service Factor	100.0	64.7	67.1
20. Unit Availability Factor	100.0	66.4	68.2
21. Unit Capacity Factor (MDC Net)	94.1	61.2	61.7
22. Unit Capacity Factor (DER Net)	93.4	60.8	60.5
23. Unit Forced Outage Rate	.0	19.9	23.1

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWN/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
15	11/04/80	S	.0	H	5		CB	PUMPXX	POWER MANUALLY REDUCED TO APPROX. 10% TO ALLOW INVESTIGATION OF P-40D REACTOR COOLANT PUMP LOW OIL LEVEL PROBLEM. NO LEAK ADDED OIL AND RETURNED TO 100% POWER OPERATIONS.
16	11/16/80	S	.0	H	5		CB	PUMPXX	POWER MANUALLY REDUCED TO APPROX. 10% POWER TO ALLOW INVESTIGATION AND REPAIR OF P-40D REACTOR COOLANT PUMP LOW OIL LEVEL PROBLEM. FOUND AND REPAIRED LEAK IN OIL LEVEL TRANSMITTER, ADDED OIL AND RETURNED TO 100% POWER OPERATIONS.

SUMMARY

THE UNIT OPERATED AT OR NEAR 100% POWER THROUGHOUT THE MONTH EXCEPT FOR POWER REDUCTIONS ON THE 4TH AND 16TH.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

MONTICELLO

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Minnesota/Wright 30 miles NW of Minneapolis
2.	Electric Reliability Council	Mid-Continent Area Reliability Coordination Agreement
3.	Type of Reactor	BWR
4.	Power Level (MWT)	1670
5.	Design Electrical Rating (MWE Net)	545
6.	Date of Initial Criticality	December 10, 1970
7.	Date of Commercial Operation	June 30, 1971
8.	Condenser Cooling Method	Once Through with Cooling Tower
9.	Condenser Cooling Water Source	Mississippi River

Utility & Contractor Information

10.	Licensee	Northern States Power Co.
11.	Corporate Address	414 Nicollet Mall - 8th Floor Minneapolis, Minnesota 55401
12.	Corporate Contact	G. H. Neils (Oper.) L. O. Mayer (Lic.)
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	GE
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. F. HEISHMAN
18.	IE Section Chief	W. S. Little
19.	IE Resident Inspector	C. Brown
20.	Licensing Project Manager	K. Eccleston
21.	Docket Number	50-263
22.	License Number/Date of Issue	DPR-22 issued 9/8/70
23.	Public Document Room	Environmental Conservation Library Minneapolis Public Library 300 Nicollet Mall Minneapolis, Minnesota 55401

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

OPERATING ROUTINELY.

+LAST IE SITE INSPECTION DATE: SEPTEMBER 2-30, 1980

+ INSPECTION REPORT NO: 80-18

SUMMARY

INSPECTION SUMMARY

+ INSPECTION ON SEPTEMBER 2-30, (80-18): REVIEW OF ROUTINE OPERATIONS, LOGS AND RECORDS, REVIEW OF RESPONSES OF IE BULLETINS AND IE CIRCULARS, REVIEW OF LER'S, HOUSEKEEPING, AND ROUTINE RADIATION PROTECTION, SURVEILLANCE, MAINTENANCE, TRAINING OF PERSONNEL, AND DESIGN CHANGE WORK PACKAGES. THE INSPECTION INVOLVED 140 INSPECTOR-HOURS ONSITE BY TWO NRC INSPECTORS INCLUDING 33 INSPECTOR-HOURS ONSITE DURING OFF-SHIFTS. NO ITEMS OF NON-COMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN THESE AREAS.

REPORTS RECEIVED FROM LICENSEE

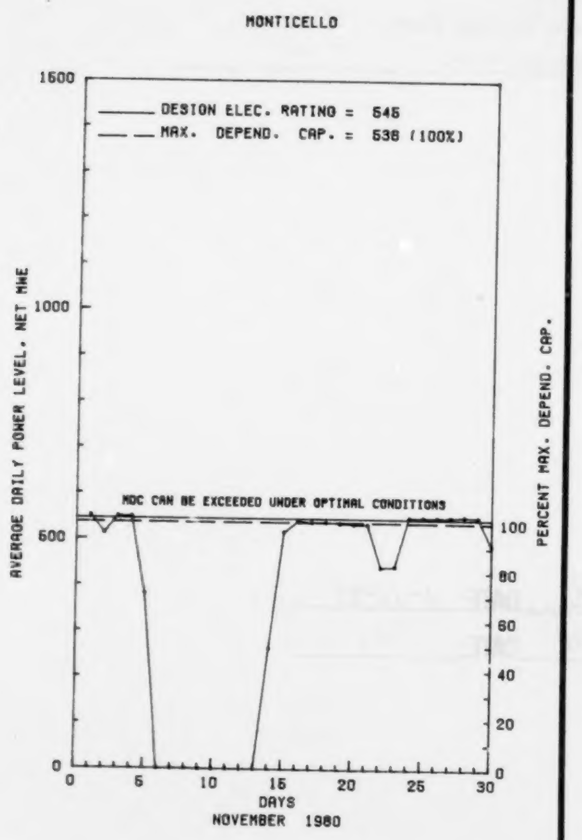
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-29/ 03L-0	09/30/80	10/30/80	MAIN STEAM DRAIN LINE LEAK.
80-30/ 03L-0	10/13/80	11/14/80	DRYWELL FLOOR DRAIN SUMP OUTBOARD ISOLATION VALVE TO CLOSE.

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 1-16-81

REVIEWED BY NRR DL: **DATE:** 1-19-81

MONTICELLO

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-263
2. Utility Contact: A. L. Myrabo (612) 295-5151
3. Licensed Thermal Power (MWt): 1670
4. Nameplate Rating (Gross MWe): 632 x 0.9 = 569
5. Design Electrical Rating (Net MWe): 545
6. Maximum Dependable Capacity (Gross MWe): 564
7. Maximum Dependable Capacity (Net MWe): 536
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	8,040.0	82,585.0
12. Hours Reactor Was Critical	538.9	6,244.1	67,775.0
13. Reactor Reserve Shutdown Hours	.0	.0	940.7
14. Hours Generator On-Line	525.1	6,133.0	66,103.1
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	816,598	9,443,177	108,386,039
17. Gross Electrical Energy (MWH)	279,160	3,194,025	34,172,085
18. Net Electrical Energy (MWH)	267,868	3,047,013	32,679,217
19. Unit Service Factor	72.9	76.3	80.0
20. Unit Availability Factor	72.9	76.3	80.0
21. Unit Capacity Factor (MDC Net)	69.4	70.7	73.8
22. Unit Capacity Factor (DER Net)	68.3	69.5	72.6
23. Unit Forced Outage Rate	1.4	8.5	6.7

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
REFUELING - 05/09/81 - 56 DAYS
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
15	11/05/80	S	187.4	B	1		ZZ	ZZZZZZ	SCHEDULED ORDERLY SHUTDOWN TO REPAIR MSIV ACTUATORS, REPLACE STEAM CHASE CABLING, INSTALL POST-LOCA CONTAINMENT RECOMBINER PENETRATIONS AND REPAIR STEAM LINE DRAINS AND FEEDWATER HEATER LEAKS.
16	11/13/80	F	7.5	A	9		CH	INSTRU	IRM HI-HI SCRAM RECEIVED FROM COLD WATER REACTIVITY INSERTION AFTER FAILURE OF LOW-FLOW FEEDWATER CONTROL VALVE CONTROLLER. THE PLANT WAS IN A SHUTDOWN CONDITION PRIOR TO THE START OF THIS OUTAGE.

SUMMARY

MONTICELLO OPERATED AT OR NEAR FULL POWER DURING NOVEMBER, WITH THE EXCEPTION OF 2 OUTAGES FOR MAINTENANCE AND EQUIPMENT FAILURE.

LEGEND

[1] Type:
F-Forced
S-Scheduled

[2] Reason:
A-Equipment Failure
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error
H-Other

[3] Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Continuations
5-Load Reduction
9-Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

NINE MILE POINT 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	New York/Oswego 8 miles NE of Oswego
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	1850
5.	Design Electrical Rating (MWE Net)	620
6.	Date of Initial Criticality	September 5, 1969
7.	Date of Commercial Operation	December 1969
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Lake Ontario

Utility & Contractor Information

10.	Licensee	Niagara Mohawk Power Corporation
11.	Corporate Address	300 Erie Boulevard West Syracuse, New York 13202
12.	Corporate Contact	R. Schneider, Vice President Electric Operations
13.	Architect/Engineer	Utility
14.	Nuclear Steam Supply System	GE
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Section Chief	H. B. KISTER
19.	IE Resident Inspector	W. BAUNACK (315) 342-4041 FTS: 264-3311 - Operator
20.	Licensing Project Manager	P. Polk
21.	Docket Number	50-220
22.	License Number/Date of Issue	DPR-63, 12/26/74
23.	Public Document Room	State University College of Oswego Penfield Library - Documents Oswego, NY 13126 (315)341-2323

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

```
    I N S P E C T I O N   S T A T U S

    OTHER ITEMS OF INTEREST

+   MANAGERIAL ITEMS:

+   MR. M. SILLIMAN HAS BEEN NAMED ACTING
    GENERAL SUPERINTENDENT FOLLOWING THE REMOVAL
    OF MR. T. PERKINS AS A RESULT OF AN NRC
    ORDER DATED NOVEMBER 26, 1980.


    PLANT STATUS:

+   THE PLANT IS OPERATING AT 92% POWER AS A
    RESULT OF A SELF IMPOSED LIMIT OF 93% DUE
    TO A POSSIBLE ERROR IN THE CORE ANALYSIS.
    REPRESENTATIVE STACK AND OFFGAS RELEASE
    RATES ARE 10 AND 13,000 MICROCURIES PER SECOND.
    RESPECTIVELY.

+ LAST IE SITE INSPECTION DATE:  11/2-30/80

+        INSPECTION REPORT NO:  50-220/80-19

    SUMMARY

    INSPECTION SUMMARY

+   50-220/80-10 - SEP 15-19: ROUTINE, UNANNOUNCED, ONSITE INSPEC-
    TION BY A REGION-BASED INSPECTOR OF LICENSEE ACTION ON PREVIOUS
    INSPECTION FINDINGS; ADMINISTRATIVE CONTROL OF SAFETY RELATED
    CALIBRATIONS; SURVEILLANCE CALIBRATIONS OF SAFETY RELATED COM-
    PONENTS AND EQUIPMENT; CALIBRATIONS OF COMPONENTS AND EQUIP-
    MENT ASSOCIATED WITH SAFETY RELATED SYSTEMS AND/OR FUNCTIONS;
    CALIBRATION AND CONTROL OF TEST EQUIPMENT; WITNESSING OF
    CALIBRATIONS; TECHNICIAN QUALIFICATIONS; AND A FACILITY TOUR.
    TWO NONCOMPLIANCES WERE IDENTIFIED: FAILURE TO COMPLETELY
    REVIEW, IDENTIFY AND CORRECT INCORRECT TEST RESULTS; FAILURE
    TO RETAIN CALIBRATION RECORDS.

+   50-220/80-14 - OCT 14-17: ROUTINE, UNANNOUNCED, ONSITE INSPEC-
    TION BY A REGION-BASED INSPECTOR (22 HRS) OF FOLLOWUP ON
    PRIOR IDENTIFIED ITEMS; ADMINISTRATIVE CONTROLS FOR SURVEIL-
    LANCE PROCEDURES; SURVEILLANCE TESTING; INSPECTOR WITNESSING
    OF SURVEILLANCE TESTS; TECHNICIAN QUALIFICATION; AND CONTROL
    ROOM AND FACILITY TOURS.  NO NONCOMPLIANCES WERE IDENTIFIED.
```

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

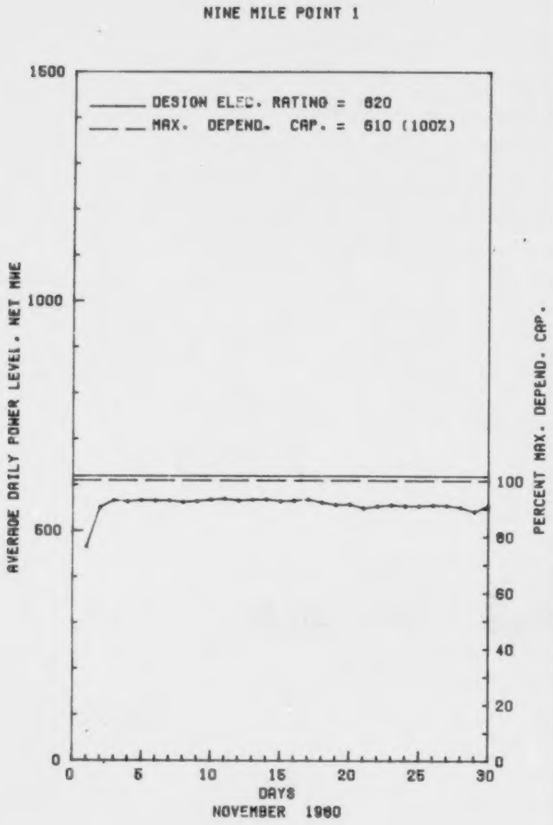
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-022/03L	09/27/80	10/24/80	LPPM DETECTOR FAILURE RENDERING APRM INOPERABLE
80-023/03L	09/22/80	10/20/80	OVERPRESSURIZED #11 EMERGENCY VENTILATION SYSTEM
80-024/	09/30/80	10/30/80	FAILURE TO RECORD TRIP POINT VALUE ON S.T.

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 1-16-81

REVIEWED BY NRR DL: **DATE:** 1/19/81

NINE MILE POINT 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-220
2. Utility Contact: J. HALLENBECK (315) 343-2110 X1312
3. Licensed Thermal Power (MWt): 1850
4. Nameplate Rating (Gross MWe): 755 x 0.85 = 642
5. Design Electrical Rating (Net MWe): 620
6. Maximum Dependable Capacity (Gross MWe): 630
7. Maximum Dependable Capacity (Net MWe): 610
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): 570
10. Reasons for Restrictions, If Any: EOC DERATE TO 93.5% CTP FOR SRITC

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	8,040.0	97,152.0
12. Hours Reactor Was Critical	720.0	7,492.7	72,845.4
13. Reactor Reserve Shutdown Hours	.0	.0	1,204.2
14. Hours Generator On-Line	720.0	7,391.1	70,201.6
15. Unit Reserve Shutdown Hours	.0	.0	20.2
16. Gross Thermal Energy (MWH)	1,231,824	12,945,708	114,687,107
17. Gross Electrical Energy (MWH)	414,752	4,298,338	37,897,848
18. Net Electrical Energy (MWH)	401,507	4,159,895	36,609,660
19. Unit Service Factor	100.0	91.9	72.3
20. Unit Availability Factor	100.0	91.9	72.3
21. Unit Capacity Factor (MDC Net)	91.4	84.8	61.8
22. Unit Capacity Factor (DER Net)	89.9	83.5	60.8
23. Unit Forced Outage Rate	.0	5.2	8.9

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): SCHED. REFUELING OUTAGE: 3-1-80 TO 6-6-80 APPROX. 14 WKS.
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
NONE									

SUMMARY

NINE MILE POINT 1 OPERATED NORMALLY DURING NOVEMBER, WITH NO OUTAGES OR REDUCTIONS REPORTED.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

NORTH ANNA 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Louisa County, Virginia 40 Miles NW of Richmond, Virginia
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	PWR
4.	Licenses Power Level (MWT)	2775
5.	Design Electrical Rating (MWE Net)	907
6.	Date of Initial Criticality	April 5, 1978
7.	Date of Commercial Operation	June 6, 1978
8.	Condenser Cooling Method	Once through
9.	Condenser Cooling Water Source	Lake Anna

Utility & Contractor Information

10.	Licensee	Virginia Electric & Power Co.
11.	Corporate Address	P. O. Box 26666 Richmond, Virginia 23261
12.	Corporate Contact	J. H. Ferguson Exec. V.P. - Power
13.	Architect/Engineer	Stone & Webster
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. Lewis
18.	IE Section Chief	P. J. Kellog
19.	IE Resident Inspector	E. H. Webster (703) 894-5421
20.	Licensing Project Manager	L. Engle
21.	Docket Number	50-338
22.	License Number/Date of Issue	NPF-4, 11/26/77
23.	Public Document Room	Alderman Library/Manuscripts Dept. Univ. of Virginia/Charlottesville VA 22901 & Louisa County Courthouse, Louisa, VA 23093

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

- ROUTINE OPERATION IN COASTDOWN MODE.
- LAST IE SITE INSPECTION DATE: OCTOBER 21-24, 1980
- INSPECTION REPORT NO: 50-338/80-36

SUMMARY

INSPECTION SUMMARY

INSPECTION OCTOBER 21-24 (80-36): THIS SPECIAL, ANNOUNCED INSPECTION INVOLVED 23 INSPECTOR-HOURS ON SITE IN THE AREAS OF INSTALLED EQUIPMENT REVIEW WITH RESPECT TO IE BULLETIN 79-01B. OF THE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATION WERE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

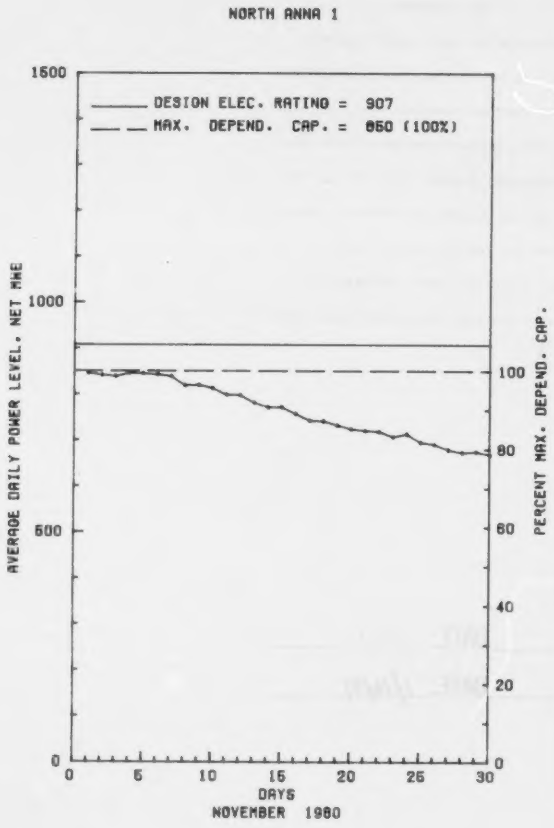
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-070/ 03L	08/14/80	08/27/80	CONTAINMENT AIR PARTICULATE HIGH FLOW ALARM ACTIVATED WHEN RELAY DISCONNECTED.
80-075/ 03L-0	09/04/80	09/18/80	B SI ACCUMULATOR BORON CONCENTRATION EXCEEDED 2100 PPM TS LIMIT.
80-079/ 03L-0	09/20/80	10/16/80	NORMALIZED AXIAL POWER DISTRIBUTION FJ2 EXCEEDED LIMIT BY 0.8%
80-081/ 03L-0	09/23/80	10/17/80	RCS RTD PROTECTION CHANNELS NOT CALIBRATED WITHIN REQUIRED SURVEILLANCE INTERVAL.
80-083/ 03L-0	09/27/80	10/24/80	PRESSURIZER LEVEL TRANSMITTTER LI-1461 INOPERABLE.
80-085/ 03L-0	10/11/80	11/03/80	PRESSURIZER LEVEL TRANSMITTER LT-1461 DECLARED INOPERABLE.
80-086/ 03L-0	10/13/80	11/03/80	AUX FEEDWATER PUMP DECLARED INOPERABLE DUE TO RELIEF VALVE FAILURE TO RESEAT.
80-087/ 03L-0	10/15/80	11/10/80	TURBINE TRIP VALVE TRIPPED ON STEAM DRIVER AUX FEEDWATER PUMP WHILE TESTING.
80-088/	10/17/80	11/10/80	AXIAL FLUX DIFFERENCE EXCEEDED +5% TARGET BAND.

REVIEWED BY OMPA LOEB: ______ **DATE:** 1/16/81

REVIEWED BY NRR DL: ______ **DATE:** 1/16/81

NORTH ANNA 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-338
2. Utility Contact: W. R. Madison (703) 894-5151
3. Licensed Thermal Power (MWt): 2775
4. Nameplate Rating (Gross MWe): 1089 x 0.9 = 980
5. Design Electrical Rating (Net MWe): 907
6. Maximum Dependable Capacity (Gross MWe): 880
7. Maximum Dependable Capacity (Net MWe): 850
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	8,040.0	21,817.0
12. Hours Reactor Was Critical	720.0	7,155.6	17,320.8
13. Reactor Reserve Shutdown Hours	.0	64.8	213.1
14. Hours Generator On-Line	720.0	6,943.2	16,996.3
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,793,316	17,682,694	43,962,517
17. Gross Electrical Energy (MWH)	584,355	5,587,018	13,933,367
18. Net Electrical Energy (MWH)	549,449	5,256,749	13,110,195
19. Unit Service Factor	100.0	86.4	77.9
20. Unit Availability Factor	100.0	86.4	77.9
21. Unit Capacity Factor (MDC Net)	89.8	76.9	70.7
22. Unit Capacity Factor (DER Net)	84.1	72.1	66.3
23. Unit Forced Outage Rate	.0	7.5	9.3

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
REFUELING, TMI & FIRE PROT. MOD.; 12/31/80 (6 WKS).
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-26	11/01/80	S	.0	H	5				BEGAN SCHEDULED UNIT COASTDOWN DUE TO END OF LIFE ON CYCLE 2 CORE.

SUMMARY

UNIT 1 ENTERED END OF LIFE ON CYCLE 2 CORE AND BEGAN A SCHEDULED POWER COASTDOWN THROUGH THE END OF THE MONTH.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

NORTH ANNA 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Louisa County, VA 40 Miles NW of Richmond, VA
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	PWR
4.	Licensed Power Level (MWT)	2775
5.	Design Electrical Rating (MWE Net)	907
6.	Date of Initial Criticality	June 12, 1980
7.	Date of Commercial Operation	N/A
8.	Condenser Cooling Method	Once through
9.	Condenser Cooling Water Source	Lake Anna

Utility and Contractor Information

10.	Licensee	Virginia Electric & Power Co.
11.	Corporate Address	P.O. Box 26666 Richmond, VA 23261
12.	Corporate Contact	J. H. Ferguson, Exec. V.P. - Power
13.	Architect/Engineer	Stone & Webster
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. Lewis
18.	IE Section Chief	P. J. Kellog
19.	IE Resident Inspector	E. H. Webster (703) 894-5421
20.	Licensing Project Manager	L. Engle
21.	Docket Number	50-339
22.	License Number/Date of Issue	NPF-7, 8/21/80
23.	Public Document Room	Alderman Library/Manuscripts Dept. Univ. of Virginia/Charlottesville, VA 22901 & Louisa County Courthouse, Louisa, VA 23093

INSPECTION STATUS

ENFORCEMENT STATUS

NONE.

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

CONTINUE POWER ESCALATION TESTING

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

- POWER ASCENSION TESTING CONTINUES.
- LAST IE SITE INSPECTION DATE: OCTOBER 21-24, 1980
- INSPECTION REPORT NO: 50-339/80-34

SUMMARY

INSPECTION SUMMARY

INSPECTION OCTOBER 21-24 (80-34): THIS SPECIAL, ANNOUNCED INSPECTION INVOLVED 28 INSPECTOR-HOURS ON SITE IN THE AREAS OF INSTALLED EQUIPMENT REVIEW WITH RESPECT TO IE BULLETIN 79-01B. OF THE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATION WERE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

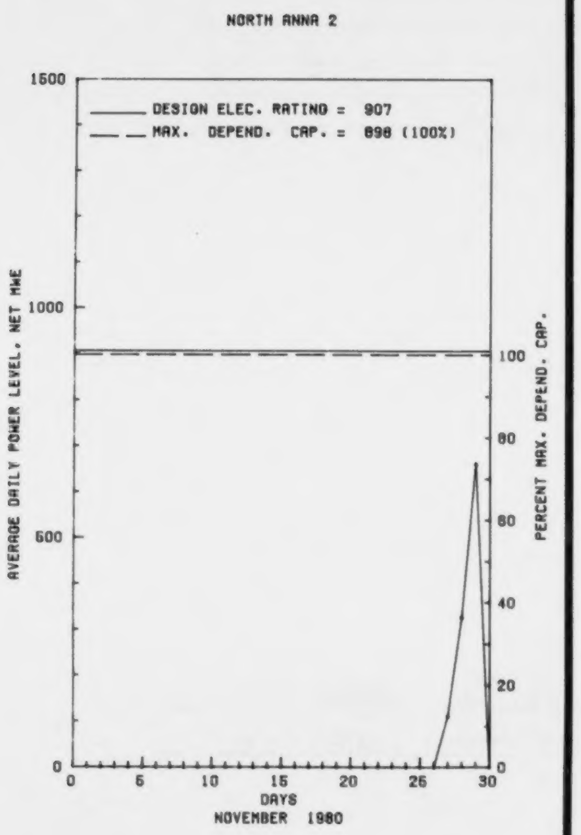
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-064/ 03L-0	09/24/80	10/23/80	INDICATED AXIAL FLUX DIFFERENCE EXCEEDED +5% TARGET BAND.
80-065/ 03L-0	10/02/80	10/30/80	PRESSURIZER BLOCK VALVE FAILED TO OPEN ON DEMAND.
80-067/ 03L-0	10/07/80	10/30/80	NEUTRON FLUX DISTRIBUTION EXCEEDED +5% AXIAL FLUX DISTRIBUTION TARGET BAND.
80-068/ 03L-0	10/04/80	11/03/80	SUMMING AMP CARD HIGH STEAM FLOW SETPOINT WAS NONCONSERVATIVELY OUT OF CALIBRATION.
80-069/ 03L-0	10/12/80	11/05/80	F(Q) LIMIT NEAR CENTER OF THE CORE WAS EXCEEDED.
80-071/ 03L-0	10/09/80	11/05/80	SI ACCUMULATORS 1A AND 1C BORON CONCENTRATION OUT OF SPEC HIGH.
80-072/ 03L-0	10/08/80	11/03/80	BORON INJECTION TANK AND BORIC ACID STORAGE TANK C OUT OF SPEC LOW.
80-073/ 03L-0	10/15/80	11/05/80	COMPARATOR CARD IN CHANNEL II LOW PRESSURE REACTOR TRIP CIRCUIT FOUND INOPERABLE.
80-074/ 03L-0	10/15/80	11/03/80	1H EMERGENCY DIESEL GENERATOR DAY TANK OUT OF SPEC LOW.
80-079/ 03L-0	10/17/80	11/10/80	NUCLEAR POWER RANGE CHANNELS INDICATED LOWER THAN DAILY CALORIMETRIC.
80-081/ 03L-0	10/21/80	11/05/80	CONTROL ROD BANK B ROD D10 AND SHUTDOWN BANK A ROD J03 DEVIATED FROM DEMAND POSITION.

REVIEWED BY OMPA LOEB: E Bole **DATE:** 1/16/81

REVIEWED BY NRR DL: JBEngle **DATE:** 1/16/81

NORTH ANNA 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-339
2. Utility Contact: W. A. WOODSMALL, III (703) 894-5151
3. Licensed Thermal Power (MWt): 2775
4. Nameplate Rating (Gross MWe): 947
5. Design Electrical Rating (Net MWe): 907
6. Maximum Dependable Capacity (Gross MWe): 928
7. Maximum Dependable Capacity (Net MWe): 898
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	6,393.0	6,393.0
12. Hours Reactor Was Critical	54.5	2,000.2	2,000.2
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	60.7	1,289.4	1,289.4
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	89,805	2,434,855	2,434,855
17. Gross Electrical Energy (MWH)	28,718	782,050	782,050
18. Net Electrical Energy (MWH)	26,376	732,929	732,929
19. Unit Service Factor			
20. Unit Availability Factor			
21. Unit Capacity Factor (MDC Net)			
22. Unit Capacity Factor (DER Net)			
23 Unit Forced Outage Rate			

NOT IN COMMERCIAL OPERATION

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): MAINTENANCE; APRIL 3, 1981; 10 DAYS
25. If Shut Down At End of Report Period, Estimated Date of Startup: 12/12/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause and Corrective Action to Prevent Recurrence
80-11	11/01/80	S	633.8	B	4				A SCHEDULED TRIP WAS CONDUCTED FROM 10% POWER IN ACCORDANCE WITH 2-SU-38 BLACKOUT TEST. COMMENCED SCHEDULED MAINTENANCE OUTAGE.
80-12	11/29/80	F	25.4	A	3				REACTOR/TURBINE TRIP ON GENERATOR LEADS DIFFERENTIAL

SUMMARY

UNIT 2 CONTINUED POWER ASCENSION TESTING. THE UNIT IS SCHEDULED TO ENTER INTO COMMERCIAL OPERATION ON THE WEEK OF DECEMBER 14, 1980.

LEGEND

[1]Type:	[2]Reason:		[3]Method:	[4]Exhibit F:	[5]Exhibit H:
F--Forced S--Scheduled	A--Equipment Failure B--Maintenance or Test C--Refueling D--Regulatory Restriction	E--Operator Training & Licensee Examination F--Administrative G--Operational Error H--Other	1--Manual 2--Manual Scram 3--Automatic Scram 4--Continuations 5--Load Reduction 9--Other	Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)	Same source

OCONEE 1

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	South Carolina/Oconee 30 miles W of Greenville
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2568
5.	Design Electrical Rating (MWE Net)	887
6.	Date of Initial Criticality	April 19, 1973
7.	Date of Commercial Operation	July 15, 1973
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Keowee Lake

Utility & Contractor Information

10.	Licensee	Duke Power Company
11.	Corporate Address	422 S. Church Street Charlotte, North Carolina 28242
12.	Corporate Contact	W. O. Parker, Jr.
13.	Architect/Engineer	DPC and Bechtel (Containment Bldg.)
14.	Nuclear Steam Supply System	B&W
15.	Constructor	DPC

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R.C. Lewis
18.	IE Section Chief	R. MARTIN
19.	IE Resident Inspector	F. Jape (803) 882-6927
20.	Licensing Project Manager	M. B. Fairtile
21.	Docket Number	50-269
22.	License Number/Date of Issue	DPR-38, 2/6/73
23.	Public Document Room	Oconee County Library 201 S. Spring Street Walhalla, South Carolina 29691

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

100% POWER.

+ LAST IE SITE INSPECTION DATE: NOVEMBER 3-28, 1980

INSPECTION REPORT NO: 50-269/80-35

SUMMARY

INSPECTION SUMMARY

+ INSPECTION SEPTEMBER 2-30 (80-32): THIS ROUTINE UNANNOUNCED INSPECTION INVOLVED 78 INSPECTOR-HOURS ON SITE IN THE AREAS OF PLANT OPERATIONS, SURVEILLANCE TESTING, REVIEW OF SPECIAL REPORT ON UNIT 2 INTEGRATED LEAK RATE TEST (ILRT), FOLLOWUP ON UNRESOLVED ITEM 287/80-21-01 AND FOLLOWUP ON INSPECTION AND ENFORCEMENT BULLETINS (IEB'S). OF THE FIVE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN FOUR AREAS; TWO ITEMS OF NONCOMPLIANCE WERE FOUND IN ONE AREA (INFRACTION - FAILURE TO COMPLY WITH TECHNICAL SPECIFICATION 3.7; AND INFRACTION - FAILURE TO COMPLY WITH TECHNICAL SPECIFICATIONS (TS) REQUIREMENT FOR PROCESSING A MAJOR PROCEDURE CHANGE).

INSPECTION OCTOBER 1-31 (80-33): THIS ROUTINE, INSPECTION INVOLVED 62 RESIDENT HOURS ON SITE IN THE AREAS OF OPERATION OBSERVATION, MAINTENANCE OBSERVATION, SURVEILLANCE OBSERVATION, ORGANIZATION AND ADMINISTRATION, LER FOLLOWUP, IEB FOLLOWUP, WITNESS OF SPENT FUEL CASK CAVITY MEASUREMENTS, AND RE-REVIEW OF CATEGORY A REQUIREMENTS OF NUREG 0578. OF THE EIGHT AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN FOUR AREAS; THREE ITEMS OF NONCOMPLIANCE WERE FOUND IN THREE AREAS (FAILURE TO COMPLY WITH TECHNICAL SPECIFICATION 4.1.2 TEST REQUIREMENT; FAILURE TO COMPLY WITH A LIMITING CONDITION FOR OPERATION AS SPECIFIED BY TECHNICAL SPECIFICATION 3.14; AND FAILURE TO FOLLOW PROCEDURE WHILE PERFORMING SURVEILLANCE RESULTING IN AN UNPLANNED RADIOACTIVE GASEOUS RELEASE).

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

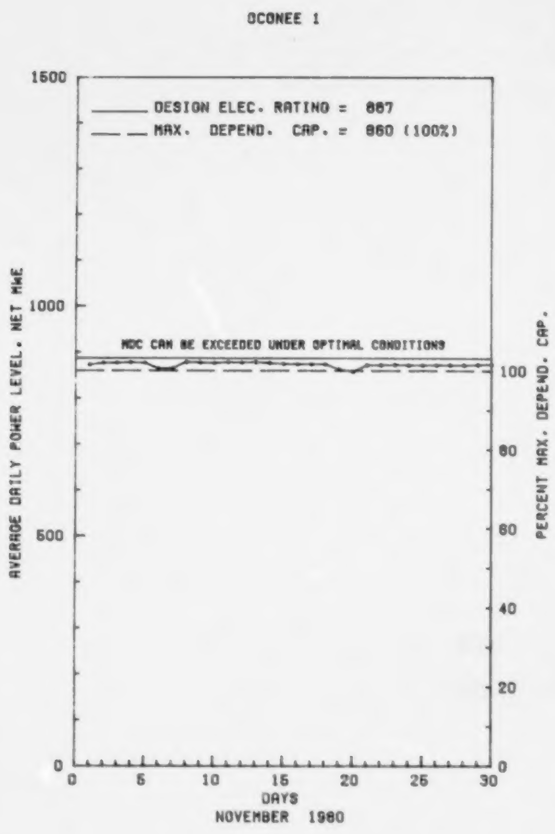
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-029/ 03L-0	09/25/80	10/24/80	KEOWEE UNIT 2 EMERGENCY POWER SUPPLY FAILED DUE TO MECHANICAL FAULT
80-030/ 03L-0	10/02/80	10/31/80	REACTOR BUILDING COOLER UNIT 1 WAS INOPERABLE DUE TO A VALVE MALFUNCTION
80-031/ 01T-0	09/02/80	10/30/80	REACTOR TRIP SETPOINT SET INCORRECTLY DUE TO PERSONNEL ERROR
80-032/ 03L-0	10/08/80	11/07/80	ONE SOURCE OF EMERGENCY POWER WAS INOPERABLE FOR ONE MINUTE
80-033/ 03L-0	10/11/80	11/10/80	THE SMOKE DETECTOR COVERING A MOTOR CONTROL CENTER WAS REMOVED FROM SERVICE FOR REPAIRS

REVIEWED BY OMPA LOEB: P E Rohe **DATE:** 1/16/81

REVIEWED BY NRR DL: M. B. Fairtile **DATE:** 1/16/81

OCONEE 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-269
2. Utility Contact: J. A. Reavis (704) 373-8552
3. Licensed Thermal Power (MWt): 2568
4. Nameplate Rating (Gross MWe): 1038 x 0.9 = 934
5. Design Electrical Rating (Net MWe): 887
6. Maximum Dependable Capacity (Gross MWe): 899
7. Maximum Dependable Capacity (Net MWe): 860
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

CAPACITY FACTORS CALCULATED WITH A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	8,040.0	64,681.0
12. Hours Reactor Was Critical	720.0	6,027.5	46,542.2
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	720.0	5,893.2	43,840.3
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,843,507	13,368,007	102,586,176
17. Gross Electrical Energy (MWH)	657,500	4,723,700	35,638,000
18. Net Electrical Energy (MWH)	628,740	4,432,107	33,713,606
19. Unit Service Factor	100.0	73.3	67.8
20. Unit Availability Factor	100.0	73.3	67.8
21. Unit Capacity Factor (MDC Net)	101.5	64.8	60.4
22. Unit Capacity Factor (DER Net)	98.4	62.8	58.8
23. Unit Forced Outage Rate	.0	11.4	17.2

24. Shutdowns Scheduled Over Next 6 Months (Type, Date and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
12-P	11/19/80	F	.0	B	5		HA	TURBIN	DID TURBINE VALVE MOVEMENT TEST TO CHECK FOR POSSIBLE BROKEN VALVE STEM. TEST INDICATED NO PROBLEM.

SUMMARY

OCONEE 1 OPERATED AT OR ABOVE FULL-POWER FOR NOVEMBER WITH 1 REDUCTION FOR TURBINE VALVE MOVEMENT TEST.

LEGEND

[1] Type
F–Forced
S–Scheduled

[2] Reason:
A–Equipment Failure
B–Maintenance or Test
C–Refueling
D–Regulatory Restriction
E–Operator Training & License Examination
F–Administrative
G–Operational Error
H–Other

[3] Method:
1–Manual
2–Manual Scram
3–Automatic Scram
4–Continuations
5–Load Reduction
9–Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

OCONEE 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	South Carolina/Oconee 30 miles W of Greenville
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2568
5.	Design Electrical Rating (MWE Net)	887
6.	Date of Initial Criticality	November 11, 1973
7.	Date of Commercial Operation	September 9, 1974
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Keowee Lake

Utility & Contractor Information

10.	Licensee	Duke Power Company
11.	Corporate Address	422 S. Church Street Charlotte, North Carolina 28242
12.	Corporate Contact	W. O. Parker, Jr.
13.	Architect/Engineer	DPC/Bechtel
14.	Nuclear Steam Supply System	B&W
15.	Constructor	DPC

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R.C. Lewis
18.	IE Section Chief	R. MARTIN
19.	IE Resident Inspector	F. Jape (803) 882-6927
20.	Licensing Project Manager	M. B. Fairtile
21.	Docket Number	50-270
22.	License Number/Date of Issue	DPR-47, 10/6/73
23.	Public Document Room	Oconee County Library 201 S. Spring Street Walhalla, South Carolina 29691

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE

FACILITY ITEMS (PLANS AND PROCEDURES):

+ NONE

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

+ SHUTDOWN FOR PLANT MODIFICATIONS.

+ LAST IE SITE INSPECTION DATE: NOVEMBER 3-28, 1980

INSPECTION REPORT NO: 50-270/80-31

SUMMARY

INSPECTION SUMMARY

+ INSPECTION SEPTEMBER 2-30 (80-28): THIS ROUTINE UNANNOUNCED INSPECTION INVOLVED 78 INSPECTOR-HOURS ON SITE IN THE AREAS OF PLANT OPERATIONS, SURVEILLANCE TESTING, REVIEW OF SPECIAL REPORT ON UNIT 2 INTEGRATED LEAK RATE TEST (ILRT), FOLLOWUP ON UNRESOLVED ITEM 287/80-21-01 AND FOLLOWUP ON INSPECTION AND ENFORCEMENT BULLETINS (IEB'S). OF THE FIVE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN FOUR AREAS; TWO ITEMS OF NONCOMPLIANCE WERE FOUND IN ONE AREA (INFRACTION - FAILURE TO COMPLY WITH TECHNICAL SPECIFICATION 3.7; AND INFRACTION - FAILURE TO COMPLY WITH TECHNICAL SPECIFICATIONS (TS) REQUIREMENT FOR PROCESSING A MAJOR PROCEDURE CHANGE).

INSPECTION OCTOBER 1-31 (80-29): THIS ROUTINE, INSPECTION INVOLVED 60 RESIDENT HOURS ON SITE IN THE AREAS OF OPERATION OBSERVATION, MAINTENANCE OBSERVATION, SURVEILLANCE OBSERVATION, ORGANIZATION AND ADMINISTRATION, LER FOLLOWUP, IEB FOLLOWUP, WITNESS OF SPENT FUEL CASK CAVITY MEASUREMENTS, AND RE-REVIEW OF CATEGORY A REQUIREMENTS OF NUREG 0578. OF THE EIGHT AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN FOUR AREAS; THREE ITEMS OF NONCOMPLIANCE WERE FOUND IN THREE AREAS (FAILURE TO COMPLY WITH TECHNICAL SPECIFICATION 4.1.2 TEST REQUIREMENT; FAILURE TO COMPLY WITH A LIMITING CONDITION FOR OPERATION AS SPECIFIED BY TECHNICAL SPECIFICATION 3.14; AND FAILURE TO FOLLOW PROCEDURE WHILE PERFORMING SURVEILLANCE RESULTING IN AN UNPLANNED RADIOACTIVE GASEOUS RELEASE).

REPORTS RECEIVED FROM LICENSEE

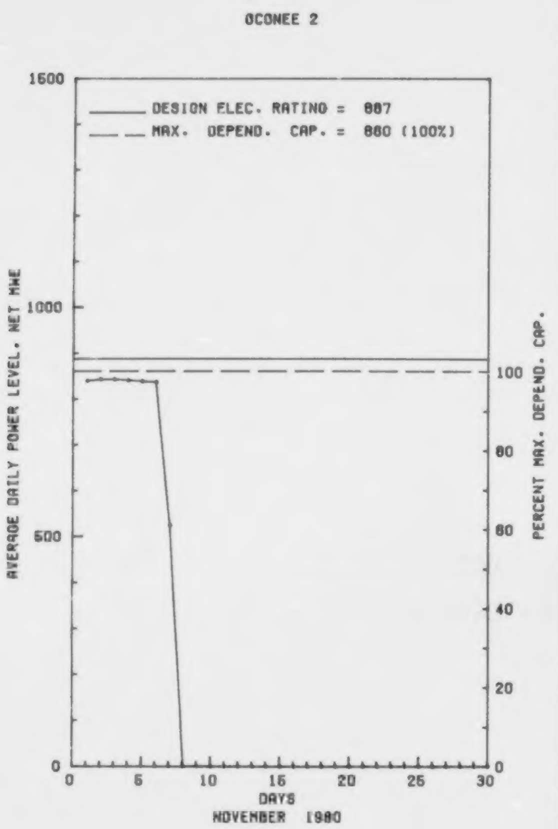
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-016/ 03L-0	09/20/80	10/20/80	THE CONCENTRATED BORIC ACID STORAGE TANK PUMP WAS INOPERABLE DUE TO A FAILURE OF THE RELIEF VALVE
80-017/ 03L-0	09/28/80	10/28/80	FAILURE OF THE CONCENTRATED BORIC ACID STORAGE TANK PUMP TO OPERATE DUE TO MECHANICAL FAULTS
80-018/ 01T-0	10/08/80	10/22/80	THE WEST PENETRATION ROOM VENTILATION SYSTEM WAS INOPERABLE DUE TO HIGH HUMIDITY
80-022/ 01T-0	10/11/80	11/06/80	AN HOURLY FIRE WATCH WAS NOT ESTABLISHED DURING MAINTENANCE ON THE CABLE SPREADING ROOM

REVIEWED BY OMPA LOEB: PE Robe **DATE:** 1/16/81

REVIEWED BY NRR DL: M. B. Fairtile **DATE:** 1/16/81

OCONEE 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-270
2. Utility Contact: J. A. Reavis (704) 373-8552
3. Licensed Thermal Power (MWt): 2568
4. Nameplate Rating (Gross MWe): 1038 x 0.9 = 934
5. Design Electrical Rating (Net MWe): 887
6. Maximum Dependable Capacity (Gross MWe): 899
7. Maximum Dependable Capacity (Net MWe): 860
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

CAPACITY FACTORS CALCULATED WITH A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	8,040.0	54,601.0
12. Hours Reactor Was Critical	162.1	4,869.8	38,465.8
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	160.6	4,766.0	37,542.1
15. Unit Reserve Shutdown Hours	.0	.0	0
16. Gross Thermal Energy (MWH)	406,572	10,487,978	88,535,382
17. Gross Electrical Energy (MWH)	139,460	3,560,110	30,074,466
18. Net Electrical Energy (MWH)	130,382	3,366,657	28,530,415
19. Unit Service Factor	22.3	59.3	68.8
20. Unit Availability Factor	22.3	59.3	68.8
21. Unit Capacity Factor (MDC Net)	21.1	48.7	60.5
22. Unit Capacity Factor (DER Net)	20.4	47.2	58.9
23. Unit Forced Outage Rate	.0	2.3	18.2

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 12/05/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
4	11/07/80	S	559.4	D	1		ZZ	ZZZZZZ	REQUIRED MODIFICATIONS NUREG-578 AND OTHER MAINTENANCE.

SUMMARY

OCONEE 2 OPERATED UNTIL NOVEMBER 7, AND ENTERED AN OUTAGE FOR REQUIRED PLANT MODIFICATIONS.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

OCONEE 3

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	South Carolina/Oconee 30 miles W of Greenville
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2568
5.	Design Electrical Rating (MWE Net)	887
6.	Date of Initial Criticality	September 5, 1974
7.	Date of Commercial Operation	December 16, 1974
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Keowee Lake

Utility & Contractor Information

10.	Licensee	Duke Power Company
11.	Corporate Address	422 S. Church Street Charlotte, North Carolina 28242
12.	Corporate Contact	W. O. Parker, Jr.
13.	Architect/Engineer	DPC/Bechtel
14.	Nuclear Steam Supply System	B&W
15.	Constructor	DPC

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R.C. Lewis
18.	IE Section Chief	R. MARTIN
19.	IE Resident Inspector	F. Jape (803) 882-6927
20.	Licensing Project Manager	M. B. Fairtile
21.	Docket Number	50-287
22.	License Number/Date of Issue	DPR-55, 7/19/74
23.	Public Document Room	Oconee County Library 201 S. Spring Street Walhalla, South Carolina 29691

INSPECTION STATUS

ENFORCEMENT STATUS

TECHNICAL SPECIFICATION 3.7 PRESCRIBES THE CONDITIONS FOR OFF-SITE AND ON-SITE ELECTRICAL POWER SOURCES FOR OPERATION OF STATION AUXILIARIES DURING REACTOR OPERATION. CONTRARY TO THE ABOVE, ON JULY 10 AND 11, 1980, UNIT 3 WAS OPERATED IN A MODE WITH ONLY CONTROL GRADE AUTOMATIC TRANSFER CAPABILITY BETWEEN NORMAL POWER SOURCE AND STARTUP POWER SOURCE. THE SAFETY GRADE AUTOMATIC TRANSFER CAPABILITY WAS OUT-OF-SERVICE. DURING THESE PERIODS MANUAL TRANSFER CAPABILITY WAS AVAILABLE AT ALL TIMES
(8025 INFRACTION)

CONTRARY TO TECHNICAL SPECIFICATION 6.1.2, CHANGE 12 TO INSTRUMENT PROCEDURE IP/0/A/3000/.2, ENCLOSUE 11.32, RECEIVED ONLY A SINGLE APPROVAL, BY A SENIOR REACTOR OPERATOR; AND WAS REVIEWED BY ONLY AN I & E TECHNICIAN WHO DID NOT RECOGNIZE THAT THE CHANGE WOULD PERMIT NONCOMPLIANCE WITH TECHNICAL SPECIFICATION 3.2.
(8025 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

- REFUELING OUTAGE SCHEDULED TO LAST 100 DAYS FROM DECEMBER 5, 1980.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

- REFUELING SCHEDULED FOR EARLY DECEMBER.
- LAST IE SITE INSPECTION DATE: NOVEMBER 3-28, 1980
- INSPECTION REPORT NO: 50-287/80-28

SUMMARY

INSPECTION SUMMARY

- INSPECTION SEPTEMBER 2-30 (80-25): THIS ROUTINE UNANNOUNCED INSPECTION INVOLVED 78 INSPECTOR-HOURS ON SITE IN THE AREAS OF PLANT OPERATIONS, SURVEILLANCE TESTING, REVIEW OF SPECIAL REPORT ON UNIT 2 INTEGRATED LEAK RATE TEST (ILRT), FOLLOWUP ON UNRESOLVED ITEM 287/80-21-01 AND FOLLOWUP ON INSPECTION AND ENFORCEMENT BULLETINS (IEB'S). OF THE FIVE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN FOUR AREAS; TWO ITEMS OF NONCOMPLIANCE WERE FOUND IN ONE AREA (INFRACTION - FAILURE TO COMPLY WITH TECHNICAL SPECIFICATION 3.7; AND INFRACTION - FAILURE TO COMPLY WITH TECHNICAL SPECIFICATIONS (TS) REQUIREMENT FOR PROCESSING A MAJOR PROCEDURE CHANGE).

INSPECTION OCTOBER 1-31 (80-26): THIS ROUTINE, INSPECTION INVOLVED 60 RESIDENT HOURS ON SITE IN THE AREAS OF OPERATION OBSERVATION, MAINTENANCE OBSERVATION, SURVEILLANCE OBSERVATION, ORGANIZATION AND ADMINISTRATION, LER FOLLOWUP, IEB FOLLOWUP, WITNESS OF SPENT FUEL CASK CAVITY MEASUREMENTS, AND RE-REVIEW OF CATEGORY A REQUIREMENTS OF NUREG 0578. OF THE EIGHT AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN FOUR AREAS; THREE ITEMS OF NONCOMPLIANCE WERE FOUND IN THREE AREAS (FAILURE TO COMPLY WITH TECHNICAL SPECIFICATION 4.1.2 TEST REQUIREMENT; FAILURE TO COMPLY WITH A LIMITING CONDITION FOR OPERATION AS SPECIFIED BY TECHNICAL SPECIFICATION 3.14; AND FAILURE TO FOLLOW PROCEDURE WHILE PERFORMING SURVEILLANCE RESULTING IN AN UNPLANNED RADIOACTIVE GASEOUS RELEASE).

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

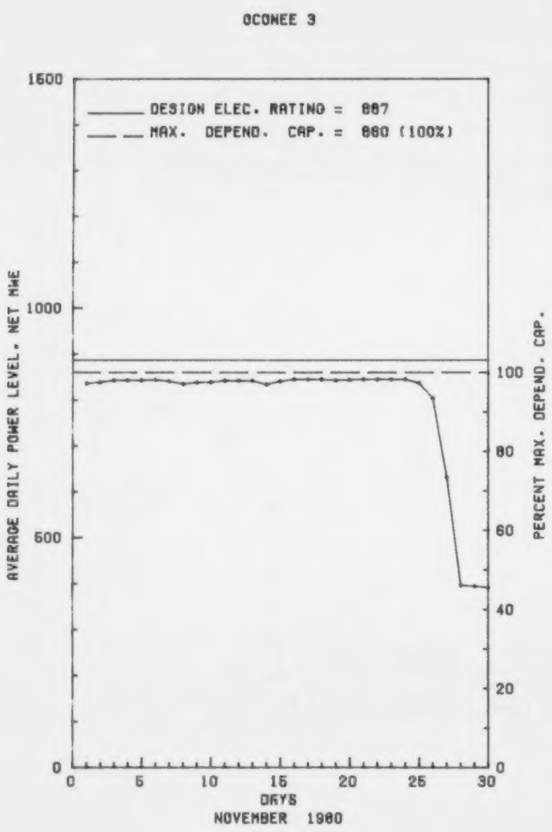
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE			

REVIEWED BY OMPA LOEB: PE Bobe **DATE:** 1/16/81

REVIEWED BY NRR DL: M.B. Fairtile **DATE:** 1/16/81

OCONEE 3

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-287
2. Utility Contact: J. A. Reavis (704) 373-8552
3. Licensed Thermal Power (MWt): 2568
4. Nameplate Rating (Gross MWe): 1038 x 0.9 = 934
5. Design Electrical Rating (Net MWe): 887
6. Maximum Dependable Capacity (Gross MWe): 899
7. Maximum Dependable Capacity (Net MWe): 860

CAPACITY FACTORS CALCULATED WITH A WEIGHTED AVERAGE

8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10 Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	8,040.0	52,248.0
12. Hours Reactor Was Critical	720.0	6,388.1	38,282.0
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	720.0	6,298.6	37,360.5
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,742,125	15,788,821	90,170,626
17. Gross Electrical Energy (MWH)	594,750	5,429,770	31,181,034
18. Net Electrical Energy (MWH)	567,872	5,174,986	29,671,542
19. Unit Service Factor	100.0	78.3	71.5
20. Unit Availability Factor	100.0	78.3	71.5
21. Unit Capacity Factor (MDC Net)	91.7	74.8	65.8
22. Unit Capacity Factor (DER Net)	88.9	72.6	64.0
23. Unit Forced Outage Rate	.0	10.8	17.0

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
REFUELING - DECEMBER 5, 1980 - 13 WEEKS
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
12P	11/25/80	S	.0	H	5		ZZ	ZZZZZZ	POWER REDUCTION TO 96% TO EXTEND REACTOR CORE LIFE. A FURTHER REDUCTION TO 55% POWER WAS MADE ON 11/27 TO EXTEND REACTOR CORE LIFE.

SUMMARY

OCONEE 3 OPERATED IN END OF CYCLE MODE ANTICIPATING REFUELING SCHEDULED FOR 12/05/80.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

OYSTER CREEK

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	New Jersey/Ocean 9 miles S of Toms River
2.	Electric Reliability Council	Mid-Atlantic Area Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	1930
5.	Design Electrical Rating (MWE Net)	650
6.	Date of Initial Criticality	May 3, 1969
7.	Date of Commercial Operation	December 1969
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Barnegat Bay

Utility & Contractor Information

10.	Licensee	Jersey Central Power & Light Co.
11.	Corporate Address	Madison Avenue At Punch Bowl Road Morristown, New Jersey 07960
12.	Corporate Contact	I. Finfrock, Vice President Power Generation
13.	Architect/Engineer	Burns & Roe
14.	Nuclear Steam Supply System	GE
15.	Constructor	J. A. Jones

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	IE Section Chief	R. R. KEIMIG
19.	IE Resident Inspector	L. BRIGGS (X5306) (FTS) 341-3000 (609) 693-0702
20.	Licensing Project Manager	W. A. Paulson
21.	Docket Number	50-219
22.	License Number/Date of Issue	DPR-16, 4/9/69
23.	Public Document Room	Ocean County Library 15 Hooper Avenue Toms River, New Jersey 08753

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

+ SYSTEM AND COMPONENT PROBLEMS:

+ PLANT SHUTDOWN ON NOVEMBER 21, 1980 TO REPAIR TUBE LEAKS IN THE 1C3 FEEDWATER HEATER AND A HINGEPIN LEAK ON A FEEDWATER CHECKVALVE.

PLANT STATUS:

+ THE PLANT OPERATED AT A NOMINAL 100% POWER DURING THE MONTH WITH OFFGAS RATE OF 83,600 MICROCURIES PER SECOND AND A STACK RELEASE RATE OF 1200 MICROCURIES PER SECOND. THE PLANT SHUTDOWN TO A COLD SHUTDOWN CONDITION AT 10:00 P.M., NOVEMBER 21, 1980 TO REPAIR TUBE LEAKS IN THE 1C3 FEEDWATER HEATER AND A HINGEPIN LEAK ON AN OUTBOARD FEEDWATER CHECK VALVE. THE PLANT RETURNED TO OPERATIONS ON NOVEMBER 29, 1980.

+ LAST IE SITE INSPECTION DATE: 11/3-28/80

+ INSPECTION REPORT NO: 50-219/80-33

SUMMARY

INSPECTION SUMMARY

+ 50-219/80-26 - JUL 14 - AUG 1: SPECIAL, ANNOUNCED INSPECTION BY TWO REGION-BASED INSPECTORS (130 HOURS) OF THE RADIATION PROTECTION PROGRAM INCLUDING: ACTIONS ON PREVIOUS INSPECTION FINDINGS AND COMMITMENTS TO THE DIRECTOR, IE, AND TO THE DIRECTOR, REGION I; RADIATION WORK PROCEDURES AND PRACTICES; PLANT DECONTAMINATION AND HOUSEKEEPING; AND RADIATION PROTECTION STAFFING. NO NONCOMPLIANCES WERE IDENTIFIED.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

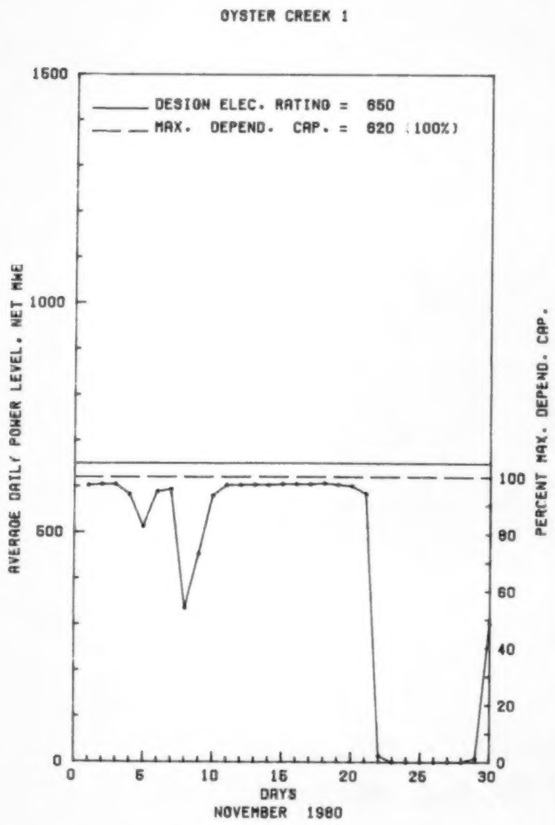
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-009-E	10/26/80	11/03/80	BLUE CRAB MORTALITY UNUSUALLY HIGH DUE TO IMPINGEMENT ON INTAKE SYSTEM DURING SEVERE WIND STORM
80-010-E	11/11/80	11/21/80	LESS THAN TWO DILUTION PUMPS IN OPERATION FOR A PERIOD OF 25 MINUTES WHEN AMBIENT WATER TEMPERATURE WAS BELOW 60 DEGREES F
80-044/03L	09/30/80	10/29/80	CORE SPRAY SYSTEM I INOPERABLE DUE TO GROUNDS ON BOOSTER PUMP ELEC. LEADS CAUSED BY ACCIDENTAL ACTUATION OF FIRE SUPPRESSION SYSTEM
80-045/03L	10/02/80	10/21/80	CORE SPRAY SYSTEM I REMOVED FROM SERVICE TO REPAIR A LEAKING VENT LINE ON THE SUCTION OF PUMP NZ01C
80-048/03L	10/29/80	11/18/80	HYDRAULIC SNUBBER NUMBER 23-7 FAILED TO LOCK UP IN THE TENSION DIRECTION
80-049/01T	11/05/80	11/20/80	'B' ISO-CONDENSER REMOVED FROM SERVICE AND ISOLATED WHEN THE CONDENSER VENT VALVES FAILED TO SHUT UPON RECEIPT OF A MANUAL ACTUATION SIGNAL

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 1-16-81

REVIEWED BY NRR DL: Walter A. Paulson **DATE:** 1-16-81

OYSTER CREEK

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-219
2. Utility Contact: C. M. McClain (201) 455-8748
3. Licensed Thermal Power (MWt): 1930
4. Nameplate Rating (Gross MWe): 687.5 x 0.8 = 550
5. Design Electrical Rating (Net MWe): 650
6. Maximum Dependable Capacity (Gross MWe): 650
7. Maximum Dependable Capacity (Net MWe): 620
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

ITEM 21 CALCULATED BY USING A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	8,040.0	95,906.0
12. Hours Reactor Was Critical	563.9	3,047.3	71,686.6
13. Reactor Reserve Shutdown Hours	.0	.0	468.2
14. Hours Generator On-Line	535.3	2,919.5	70,225.0
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	926,688	4,942,321	118,821,479
17. Gross Electrical Energy (MWH)	302,270	1,593,960	40,476,275
18. Net Electrical Energy (MWH)	236,410	1,522,558	39,003,571
19. Unit Service Factor	74.3	36.3	73.2
20. Unit Availability Factor	74.3	36.3	73.2
21. Unit Capacity Factor (MDC Net)	66.4	30.5	67.1
22. Unit Capacity Factor (DER Net)	63.3	29.1	62.6
23. Unit Forced Outage Rate	25.7	12.2	6.7

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): 04/30/81
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
6	11/07/80	F	.0	B	5		ZZ	ZZZZZZ	LEAKING DRAIN VALVES ON "C" FEEDWATER PUMP AND A FLOW CONTROL VALVE REPLACED. MAINTENANCE PERFORMED ON #4 TURBINE STOP VALVE AND 1B2 MOISTURE REMOVAL VALVE.
7	11/21/80	F	184.7	B	1		ZZ	ZZZZZZ	PLUGGED 27 LEAKING TUBES IN 1C3 HP FEEDWATER HEATER AND REPAIRED SEAL WELD ON FEED CHECK VALVE CHECK VALVE HINGE PIN.

SUMMARY

DURING THE BEGINNING OF THE MONTH, OYSTER CREEK WAS OPERATING AT 96.3% CAPACITY. LOAD WAS LIMITED BY CONDENSATE DEMINERALIZER DIFFERENTIAL PRESSURE. THE ULTIMATE CAUSE WAS INABILITY OF THE NEW RADWASTE SYSTEM TO PROCESS LIQUID AND SOLID WASTE AS DESIGNED. LOAD WAS REDUCED ON 11/4 FOR AN ABRO ON A CONDENSATE DEMINERALIZER AND ON 11/7 FOR MAINTENANCE ON "C" FEEDWATER STRING AND #4 TURBINE STOP VALVE. THE PLANT WAS SHUTDOWN ON 11/21. RESTART WAS DELAYED ONE DAY DUE TO CONTAINMENT INERTION PROBLEMS.

LEGEND

[1] Type:
F-Forced
S-Scheduled

[2] Reason:
A-Equipment Failure
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error
H-Other

[3] Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Continuations
5-Load Reduction
9-Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

PALISADES

NRC POWER RESTRICTIONS: 83% OF THERMAL POWER

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Michigan/Van Buren 5 miles S of South Haven
2.	Electric Reliability Council	East Central Area Reliability Coordination Agreement
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2530
5.	Design Electrical Rating (MWE Net)	805
6.	Date of Initial Criticality	May 24, 1971
7.	Date of Commercial Operation	December 31, 1971 (ALSO DATE OF FIRST ELEC.)
8.	Condenser Cooling Method	Forced Draft Cooling Towers
9.	Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10.	Licensee	Consumers Power Company
11.	Corporate Address	212 W. Michigan Avenue Jackson, Michigan 49201
12.	Corporate Contact	R. B. DeWitt
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Combustion Engineering
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	D.C. Boyd
19.	IE Resident Inspector	[illegible]rgenson (616) 764-8971
20.	Licensing Project Manager	R. [illegible] Silver
21.	Docket Number	50-255
22.	License Number/Date of Issue	DPR-20, 3/24/71
23.	Public Document Room	Kalamazoo Public Library 315 South Rose Street Reference Department Kalamazoo, Michigan 49006

INSPECTION STATUS

ENFORCEMENT STATUS

TECHNICAL SPECIFICATION 4.11.4 STATES THAT AQUATIC SAMPLES SHALL BE TAKEN ACCORDING TO THE FOLLOWING SCHEDULE: (B) MIXING BASIN WATER TEMPERATURE PRIOR TO DISCHARGE TO THE LAKE - MEASURE CONTINUOUSLY DURING COOLING SYSTEM OPERATION AND RECORD HOURLY. CONTRARY TO THE ABOVE, NO HOURLY RECORDINGS OF WATER TEMPERATURE PRIOR TO DISCHARGE TO THE LAKE WERE PERFORMED DURING THE PERIOD APRIL 28, 1980 THROUGH JULY 31, 1980.
(8019 DEFICIENCY)

TECHNICAL SPECIFICATION 6.9.3.1.C STATES, IN PART, THAT AN ANNUAL REPORT OF THE RESULTS OF THE AQUATIC, TERRESTRIAL, AND METEOROLOGICAL ENVIRONMENTAL MONITORING PROGRAM PERFORMED DURING THE PREVIOUS TWELVE MONTHS OF OPERATION SHALL BE SUBMITTED TO THE NRC WITHIN 90 DAYS AFTER JANUARY 1 OF EACH PRECEDING CALENDAR YEAR. CONTRARY TO THE ABOVE, THE REPORT FOR 1979 WAS NOT SUBMITTED UNTIL OCTOBER 23, 1980.
(8019 DEFICIENCY)

TECHNICAL SPECIFICATION 6.9.3.C.1. STATES, IN PART, THAT THE NONRADIOLOGICAL ENVIRONMENTAL MONITORING PROGRAM REPORT SHALL INCLUDE DATA AND NARRATIVE SUMMARIES, INTERPRETATIONS, ... OF THE NONRADIOLOGICAL ENVIRONMENTAL MONITORING PROGRAMS REQUIRED IN SECTION 4.11. FOR EACH MEDIUM SAMPLED DURING THE YEAR, A LIST AND MAP OF THE SAMPLING LOCATIONS AND TOTAL NUMBER OF SAMPLES SHALL BE INCLUDED. CONTRARY TO THE ABOVE, THE 1979 NONRADIOLOGICAL ENVIRONMENTAL MONITORING REPORT SUBMITTED DID NOT CONTAIN NARRATIVE SUMMARIES, A LIST AND MAP OF SAMPLING LOCATIONS, AND THE RESULTS OF ALL PROGRAMS REQUIRED IN SECTION 4.11.
(8019 DEFICIENCY)

TECHNICAL SPECIFICATION 6.9.3.1.A.2.C(4) STATES, IN PART, THAT "TOTAL CURIES FOR EACH OF THE RADIONUCLIDES IN PARTICULATE FORM DETERMINED TO BE RELEASED BASED ON ANALYSES PERFORMED" SHALL BE INCLUDED IN A SEMIANNUAL REPORT ON THE RADIOACTIVE DISCHARGES RELEASED FROM THE SITE. CONTRARY TO THE ABOVE, THE LICENSEE HAS NOT ACCURATELY REPORTED THE CURIE CONTENT OF THE PARTICULATES RELEASED FROM THE SITE.
(8019 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

+ PLANT IN HEATUP IN PREPARATION FOR RESTART; PLANT WAS SHUTDOWN ON OCTOBER 31, 1980, DUE TO HIGH VIBRATIONS ON THE MAIN TURBINE.

+LAST IE SITE INSPECTION DATE: OCTOBER 20-24, 1980

+ INSPECTION REPORT NO: 80-19

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-32/ 03L-0	08/26/80	09/23/80	DURING HOT SHUTDOWN, PRIMARY DATA LOGGER CEASED PERFORMING ROD SCANS AND THE SECONDARY DATA LOGGER TYPER BECAME STUCK.
80-43/ 01T-0	11/06/80	11/25/80	DURING VALVE ISI TESTING THE MEASURED LEAKAGE THROUGH CONTAINMENT PENETRATION NO. 14 EXCEEDED LIMIT OF TS 4.5.2.

SUMMARY

INSPECTION SUMMARY

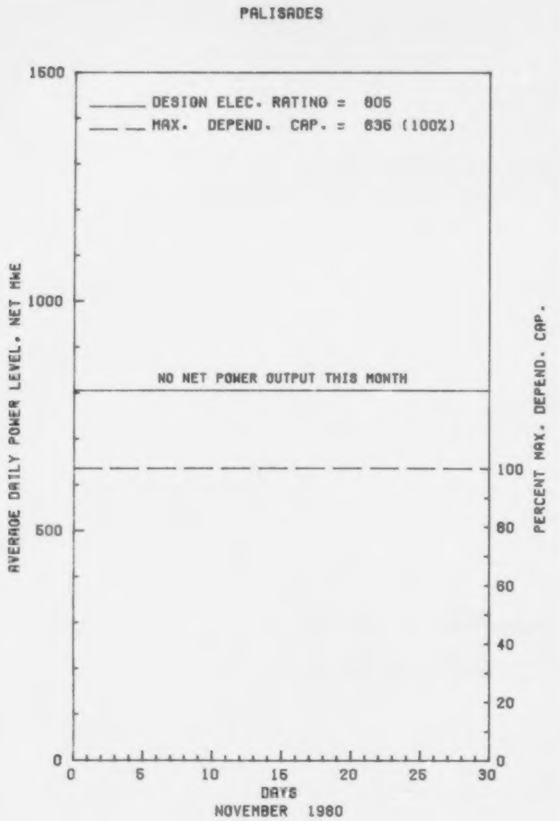
+ INSPECTION ON SEPTEMBER 1-5, 8-12, 15-19, 21-27, 29-30 AND OCTOBER 1-3, (80-18): ROUTINE MONTHLY RESIDENT INSPECTION PROGRAM ACTIVITIES INCLUDING, OPERATIONAL SAFETY; MAINTENANCE; SURVEILLANCE; FACILITY CHANGES; PROCEDURES; PLANT TRIP; CURRENT REGULATORY ITEMS; FOLLOWUP ON PREVIOUSLY IDENTIFIED ITEMS; AND OPERATOR STRIKE. THE INSPECTION INVOLVED 247 INSPECTOR-HOURS ONSITE BY THREE NRC INSPECTORS INCLUDING 24 INSPECTOR-HOURS ONSITE DURING OFF-SHIFTS. NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN ANY OF THE EIGHT AREAS INSPECTED.

+ INSPECTION ON OCTOBER 20-24, (80-19): ROUTINE, UNANNOUNCED INSPECTION OF: (1) ENVIRONMENTAL PROTECTION - INCLUDING QUALITY CONTROL OF ANALYTICAL MEASUREMENTS: PROGRAM MANAGEMENT AND IMPLEMENTATION; REVIEW OF PROGRAM RESULTS; INSPECTION OF SELECTED SAMPLING STATIONS; NONRADIOACTIVE EFFLUENT RELEASE RATES AND LIMITS; AND (2) CONFIRMATORY MEASUREMENTS - INCLUDING DISCUSSION OF PREVIOUS SAMPLE RESULTS; COLLECTION OF SAMPLES, ANALYSIS ONSITE WITH THE REGION III MEASUREMENTS VAN AND DISCUSSION OF RESULTS; AND PROGRAM FOR QUALITY ASSURANCE AND QUALITY CONTROL OF ANALYTICAL MEASUREMENTS. THE INSPECTION INVOLVED 72 INSPECTOR-HOURS ONSITE BY TWO NRC INSPECTORS. OF THE EIGHT AREAS INSPECTED, NO APPARENT ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN SIX AREAS; FOUR APPARENT ITEMS OF NONCOMPLIANCE WERE IDENTIFIED IN TWO AREAS (FAILURE TO SUBMIT THE ANNUAL NONRADIOLOGICAL ENVIRONMENTAL REPORT FOR CY79 ON TIME; FAILURE TO INCLUDE ALL INFORMATION REQUIRED IN THE ANNUAL NONRADIOLOGICAL ENVIRONMENTAL REPORT FOR CY79; FAILURE TO RECORD AND MEASURE DISCHARGE TEMPERATURE AS REQUIRED; AND FAILURE TO ACCURATELY REPORT THE TOTAL CURIES FOR EACH OF THE RADIONUCLIDES RELEASED IN PARTICULATE FORM)

REVIEWED BY OMPA LOEB: MR Beebe **DATE:** 01/16/81
REVIEWED BY NRR DL: T.V. Wambach **DATE:** 01/16/81

PALISADES

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-255
2. Utility Contact: P. J. Rodtke (616) 764-8913
3. Licensed Thermal Power (MWt): 2530
4. Nameplate Rating (Gross MWe): 955 x 0.85 = 812
5. Design Electrical Rating (Net MWe): 805
6. Maximum Dependable Capacity (Gross MWe): 675
7. Maximum Dependable Capacity (Net MWe): 635
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	8,040.0	78,471.0
12. Hours Reactor Was Critical	.0	3,467.3	43,032.1
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	.0	3,346.5	40,773.0
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	7,271,476	78,752,976
17. Gross Electrical Energy (MWH)	0	2,247,680	24,279,910
18. Net Electrical Energy (MWH)	0	2,111,648	22,782,423
19. Unit Service Factor	.0	41.6	52.0
20. Unit Availability Factor	.0	41.6	52.0
21. Unit Capacity Factor (MDC Net)	.0	41.4	45.7
22. Unit Capacity Factor (DER Net)	.0	32.6	36.1
23. Unit Forced Outage Rate	.0	10.9	34.7

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 12/12/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
20	10/31/80	S	720.0	B	4				TURBINE INSPECTION.

SUMMARY

PALISADES REMAINED OFF-LINE FOR NOVEMBER IN A CONTINUING TURBINE INSPECTION.

LEGEND

[1] Type:
- F-Forced
- S-Scheduled

[2] Reason:
- A-Equipment Failure
- B-Maintenance or Test
- C-Refueling
- D-Regulatory Restriction
- E-Operator Training & License Examination
- F-Administrative
- G-Operational Error
- H-Other

[3] Method:
- 1-Manual
- 2-Manual Scram
- 3-Automatic Scram
- 4-Continuations
- 5-Load Reduction
- 9-Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

PEACH BOTTOM 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Pennsylvania/York 19 miles S of Lancaster
2.	Electric Reliability Council	Mid-Atlantic Area Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	3293
5.	Design Electrical Rating (MWE Net)	1065
6.	Date of Initial Criticality	September 16, 1973
7.	Date of Commercial Operation	July 5, 1974
8.	Condenser Cooling Method	Once Through with Cooling Towers
9.	Condenser Cooling Water Source	Conowingo Pond/Susquehanna River

Utility & Contractor Information

10.	Licensee	Philadelphia Electric Company
11.	Corporate Address	2301 Market Street Philadelphia, Pennsylvania 19101
12.	Corporate Contact	S. L. Daltroff, Vice President Electric Production
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	GE
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	IE Section Chief	E. McCabe
19.	IE Resident Inspector	C. Cowgill 717-456-7614
20.	Licensing Project Manager	D. Verrelli
21.	Docket Number	50-277
22.	License Number/Date of Issue	DPR-44, 8/8/73
23.	Public Document Room	Government Publications Section State Library of Pennsylvania Education Building Commonwealth and Walnut Street Harrisburg, Pennsylvania 17126

INSPECTION STATUS

ENFORCEMENT STATUS

TECHNICAL SPECIFICATION 3.7.C. SECONDARY CONTAINMENT STATES IN PART, "SECONDARY CONTAINMENT INTEGRITY SHALL BE MAINTAINED DURING ALL MODES OF OPERATION...." CONTRARY TO THE ABOVE ON SEPTEMBER 23, 1980, WITH THE UNIT 2 REACTOR IN POWER OPERATION, AN INDIVIDUAL BREACHED SECONDARY CONTAINMENT AT THE NORTHEAST AIRLOCK ENTRANCE TO THE UNIT 2 REACTOR BUILDING (SECONDARY CONTAINMENT), BY BLOCKING THE INNER SECONDARY CONTAINMENT DOOR OPEN AND THEN OPENING THE OUTER SECONDARY CONTAINMENT DOOR TO REMOVE SCAFFOLDING MATERIAL FROM THE UNIT 2 REACTOR BUILDING. (8029 INFRACTION)

FAILURE TO MAINTAIN PROPER ESCORT REQUIREMENTS. (8029 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

+ SYSTEMS AND COMPONENT PROBLEMS:

+ STANDBY GAS TREATMENT SYSTEM AIR SUPPLY TO MOISTURE CONTROLLER WAS DETERMINED TO BE NOT SEISMICALLY QUALIFIED. ELECTRICAL MODIFICATION MADE TO CORRECT THE PROBLEM.
PLANT STATUS:

+ THE REACTOR WAS OPERATED AT NOMINAL FULL POWER. ON NOVEMBER 14 A 3-DAY OUTAGE BEGAN TO REPAIR A HIGH PRESSURE COOLANT INJECTION SYSTEM VALVE PACKING LEAK. A SCRAM OCCURRED DURING POWER ASCENTION ON NOVEMBER 17 DUE TO A RECIRCULATION PUMP FLOW CONTROL FAILURE. UNIT OFFGAS WAS 3295 MICROCURIES PER SECOND AND COMBINED UNIT 2 AND UNIT 3 STACK GAS RELEASE RATE WAS 443 MICROCURIES PER SECOND.

+ LAST IE SITE INSPECTION DATE: 11/8-30/80

+ INSPECTION REPORT NO: 50-277/80-33

SUMMARY

INSPECTION SUMMARY

+ 50-277/80-31 - SEP 30 - OCT 2-6: ROUTINE, UNANNOUNCED, ONSITE INSPECTION BY A REGION-BASED INSPECTOR (26 HRS) AND 18 INSPECTOR-HOURS AT THE CORPORATE OFFICE BY A REGION-BASED INSPECTOR AND A RESIDENT INSPECTOR OF QUALITY ASSURANCE PROGRAM (QAP) IMPLEMENTATION INCLUDING: AUDITING, DESIGN CHANGES/MODIFICATIONS; AND, FOLLOWUP ON INSPECTOR IDENTIFIED ITEMS. NO NONCOMPLIANCES WERE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

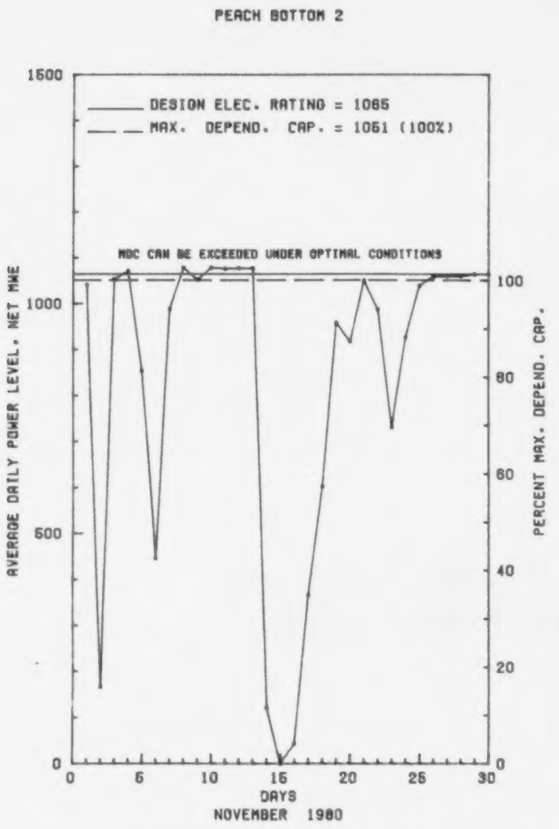
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-015/ 03L	09/04/80	09/24/80	REACTOR CORE ISOLATION COOLING SYSTEM ISOLATION VALVE FAILED TO CLOSE WHEN A LIMITORQUE OPERATOR FAILED TO FUNCTION. TEMPORARY REPAIRS WERE MADE
80-016/ 03L	09/09/80	09/29/80	ECCS A CHANNEL LOGIC RELAY FAILED TO FUNCTION DUE TO DIRTY CONTACTS. CONTACTS WERE CLEANED AND THE SYSTEM WAS RETESTED SATISFACTORILY
80-017/ 03L	09/06/80	09/30/80	B RHR SUBSYSTEM INOPERABLE DUE TO PUMP ROOM COOLER FANS FAILED TO START. THE B RHR PUMP BREAKER WAS REPLACED AND SYSTEM RESTORED
80-019/ 03L	09/06/80	10/06/80	PRIMARY CONTAINMENT GAS SAMPLE FLOW STOPPED DUE TO FAULTY SOLENOID VALVE. SOLENOID VALVE COIL WAS REPLACED AND TESTED SATISFACTORILY
80-020/ 01T	09/26/80	10/10/80	B STANDBY LIQUID CONTROL PUMP BREAKER ACCIDENTLY TRIPPED. BREAKER WAS RECLOSED.
80-026/ 01T	11/05/80	11/19/80	HIGH PRESSURE COOLANT INJECTION SYSTEM INOPERABLE DUE TO FAILED SPEED CONTROL. THE SPEED CONTROLLER WAS REPAIRED AND HPCI WAS OPERABLE 15 HOURS LATER

REVIEWED BY OMPA LOEB: **DATE:** 1-16-81

REVIEWED BY NRR DL: **DATE:** 1-17-81

PEACH BOTTOM 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-277
2. Utility Contact: W. M. Alden (215) 841-5022
3. Licensed Thermal Power (MWt): 3293
4. Nameplate Rating (Gross MWe): 1280 x 0.9 = 1152
5. Design Electrical Rating (Net MWe): 1065
6. Maximum Dependable Capacity (Gross MWe): 1098
7. Maximum Dependable Capacity (Net MWe): 1051
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	8,040.0	56,134.0
12. Hours Reactor Was Critical	669.9	4,093.9	41,848.9
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	632.7	3,839.0	40,821.8
15. Unit Reserve Shutdown Hours	.0	0	.0
16. Gross Thermal Energy (MWH)	1,935,847	11,457,326	118,222,970
17. Gross Electrical Energy (MWH)	649,400	3,815,220	38,873,450
18. Net Electrical Energy (MWH)	623,838	3,632,242	37,248,027
19. Unit Service Factor	87.9	47.7	72.7
20. Unit Availability Factor	87.9	47.7	72.7
21. Unit Capacity Factor (MDC Net)	82.4	43.0	63.1
22. Unit Capacity Factor (DER Net)	81.4	42.4	62.3
23. Unit Forced Outage Rate	12.1	3.6	6.0

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
12	11/05/80	F	12.6	A	1		HA	VALVEX	ELECTRO-HYDRAULIC CONTROL LEAK ON #1 CONTROL VALVE
13	11/14/80	F	48.3	A	3		SD	VALVEX	FEEDWATER LEAK AT HIGH PRESSURE COOLANT INJECTION TESTABLE CHECK VALVE
14	11/17/80	F	26.4	A	3		IA	INSTRU	APRM HI-FLUX AUTOMATIC SCRAM DUE TO '2A' RECIRC. FLOW CONTROL SYSTEM FAILURE
15	11/22/80	S	.8	H	5		RC	ZZZZZZ	LOAD DROP FOR CONTROL ROD SEQUENCE EXCHANGE

SUMMARY

PEACH BOTTOM 2 OPERATED WITH 3 OUTAGES FOR EQUIPMENT FAILURE, AND 1 REDUCTION FOR CONTROL ROD CHANGES.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

PEACH BOTTOM 3

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Pennsylvania/York 19 miles S of Lancaster
2.	Electric Reliability Council	Mid-Atlantic Area Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	3293
5.	Design Electrical Rating (MWE Net)	1065
6.	Date of Initial Criticality	August 7, 1974
7.	Date of Commercial Operation	December 23, 1974
8.	Condenser Cooling Method	Once Through with Cooling Towers
9.	Condenser Cooling Water Source	Conowingo Pond/Susquehanna River

Utility & Contractor Information

10.	Licensee	Philadelphia Electric Company
11.	Corporate Address	2301 Market Street Philadelphia, Pennsylvania 19101
12.	Corporate Contact	S. L. Daltroff, Vice President Electric Production
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	GE
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	IE Section Chief	E. McCabe
19.	IE Resident Inspector	C. Cowgill (717-456-7614)
20.	Licensing Project Manager	D. Verrelli
21.	Docket Number	50-278
22.	License Number/Date of Issue	DPR-56, 7/2/74
23.	Public Document Room	Government Publications Section State Library of Pennsylvania Education Building Commonwealth and Walnut Street Harrisburg, Pennsylvania 17126

INSPECTION STATUS

ENFORCEMENT STATUS

10 CFR 50 APPX B, CRITERION XVI, STATES IN PART: "MEASURES SHALL BE ESTABLISHED TO ENSURE THAT CONDITIONS ADVERSE TO QUALITY SUCH AS DEFICIENCIES, ARE PROMPTLY CORRECTED. CONTRARY TO THE ABOVE, A BROKEN FIRESEAL FOR SAFETY RELATED PENETRATION ZB3MV05 IN THE UNIT 3 REACTOR BUILDING (165 FOOT ELEVATION) WAS FOUND ON APRIL 21, 1980 TO BE MISSING ITS FIRE RETARDANT SEAL AND NO CORRECTIVE ACTION HAD BEEN TAKEN. (8007 INFRACTION)

T.S. 6.11 REQUIRES PROCEDURES FOR RADIATION PROTECTION TO BE PREPARED CONSISTENT WITH 10 CFR 20 AND THAT THOSE PROCEDURES BE APPROVED, MAINTAINED, AND ADHERED TO. H.P. PROCEDURE HPO/CO-10A, REVISION 2, "CONDUCT IN CONTROLLED AREAS MINIMIZE EXPOSURE" DATED MAY 10, 1978, STATES IN PART ..."EATING, SMOKING AND DRINKING ARE NOT PERMITTED IN THE GENERAL PLANT AREAS." SPECIFIC DESIGNATED EATING AND SMOKING AREAS HAVE BEEN IDENTIFIED. CONTRARY TO THE ABOVE, DURING INSPECTIONS CONDUCTED ON APRIL 14, 21, AND 22, 1980, THE INSPECTOR IDENTIFIED EVIDENCE OF SMOKING, INCLUDING CRUSHED PARTIALLY SMOKED CIGARETTS, USED MATCHES, PARTIALLY SMOKED CIGARS AS WELL AS TRASH, LITTER AND DEBRIS IN THE FOLLOWING NONDESIGNATED SMOKING AREAS: TURBINE BLDG, UNIT 3 ELECTRICAL PANEL 30G21, BEHIND SAFETY RELATED CABLE TRAY ZB3N42A, IN UNIT 3 FINAL POOL COOLING DRAINS, THE CABLE SPREADING ROOM, IN THE OVERHEAD ABOVE THE CONTROL ROOM, ETC. (8007 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEM AND COMPONENT PROBLEMS:

- STANDBY GAS TREATMENT SYSTEM AIR SUPPLY TO MOISTURE CONTROLLER WAS DETERMINED TO BE NOT SEISMICALLY QUALIFIED. ELECTRICAL MODIFICATION MADE TO CORRECT PROBLEM.
- A CRACK IN THE REACTOR WATER CLEANUP SYSTEM SUCTION LINE WAS DISCOVERED NOVEMBER 24. REPAIRS ARE IN PROGRESS.

PLANT STATUS:

- REACTOR OPERATED AT NOMINAL FULL POWER. UNIT OFFGAS RATE WAS 3464 MICROCURIES PER SECOND AND COMBINED UNIT 2 AND UNIT 3 STACK GAS RELEASE RATE WAS 443 MICROCURIES PER SECOND.

- LAST IE SITE INSPECTION DATE: 11/8-30/80
- INSPECTION REPORT NO: 50-278/80-26

SUMMARY

INSPECTION SUMMARY

- 50-278/80-23 - SEP 30 - OCT 2-6: ROUTINE, UNANNOUNCED, ONSITE INSPECTION BY A REGION-BASED INSPECTOR (26 HRS) AND 18 INSPECTOR-HOURS AT THE CORPORATE OFFICES BY ONE REGION-BASED INSPECTOR AND A RESIDENT INSPECTOR OF QUALITY ASSURANCE PROGRAM (QAP) IMPLEMENTATION INCLUDING: AUDITING, DESIGN CHANGES/ MODIFICATIONS; AND FOLLOWUP ON INSPECTOR IDENTIFIED ITEMS. NO NONCOMPLIANCES WERE IDENTIFIED.
- 50-278/80-25 - OCT 29: ROUTINE, ANNOUNCED, ONSITE INSPECTION BY A REGION-BASED INSPECTOR (8 HRS) OF MOTOR OPERATED VALVES AND AIR OPERATED VALVES OF THE HIGH PRESSURE COLLANT INJECTION SYSTEM TO VERIFY COMPONENT INSTALLATION AS IDENTIFIED AND REPORTED BY THE LICENSEE IN RESPONSE TO NRC BULLETIN NO. 79-01B. NO NONCOMPLIANCES WERE IDENTIFIED.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-022/ 03L	08/29/80	09/19/80	3B CORE SPRAY PUMP STARTED .3 SECONDS TOO SOON ON A SIMULATED LOSS OF POWER TIME DELAY. RELAY WAS ADJUSTED AND RETESTED SATISFACTORILY
80-023/ 03L	09/08/80	09/30/80	A TEMPERATURE SWITCH ON REACTOR WATER CLEANUP SYSTEM FAILED TO ACTUATE AT THE PROPER TEMPERATURE DURING SURVEILLANCE TESTING. THE SWITCH WAS RECALIBRATED AND RETESTED SATISFACTORILY
80-024/ 03X	09/11/80	10/22/80	PRIMARY CONTAINMENT GAS SAMPLE FLOW STOPPED DUE TO OPEN CIRCUIT IN PUMP LOCAL CONTROL SWITCH. SWITCH REPLACED AND PUMP TESTED SATISFACTORILY
80-026/ 01T	11/14/80	11/26/80	HIGH PRESSURE COOLANT INJECTION SYSTEM WAS INOPERABLE FOR ABOUT FOUR HOURS WHEN THE PUMP FLOW CONTROLLER FAILED. THE CONTROLLER WAS REPAIRED AND REPLACED. HPCI WAS TESTED SATISFACTORILY

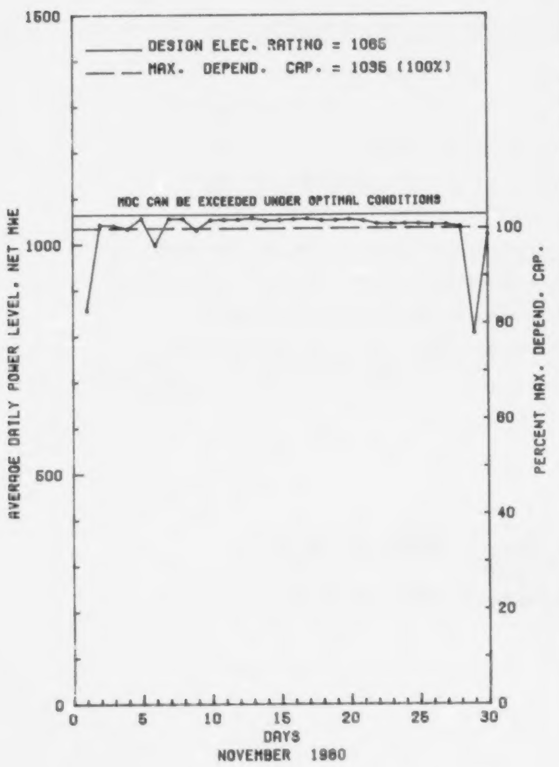
REVIEWED BY OMPA LOEB: **DATE:** 1-16-81

REVIEWED BY NRR DL: **DATE:** 1-17-81

PEACH BOTTOM 3

AVERAGE DAILY POWER LEVEL (MWe)

PEACH BOTTOM 3



OPERATING STATUS

1. Docket Number: 50-278
2. Utility Contact: W. M. Alden (215) 841-5022
3. Licensed Thermal Power (MWt): 3293
4. Nameplate Rating (Gross MWe): 1280 x 0.9 = 1152
5. Design Electrical Rating (Net MWe): 1065
6. Maximum Dependable Capacity (Gross MWe): 1098
7. Maximum Dependable Capacity (Net MWe): 1035
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	8,040.0	52,080.0
12. Hours Reactor Was Critical	720.0	6,667.0	41,513.0
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	720.0	6,461.8	40,393.6
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	2,320,694	20,592,130	114,811,284
17. Gross Electrical Energy (MWH)	771,890	6,837,400	37,423,260
18. Net Electrical Energy (MWH)	742,603	6,581,584	35,926,386
19. Unit Service Factor	100.0	80.4	77.6
20. Unit Availability Factor	100.0	80.4	77.6
21. Unit Capacity Factor (MDC Net)	99.7	79.1	66.7
22. Unit Capacity Factor (DER Net)	96.8	76.9	64.8
23. Unit Forced Outage Rate	.0	10.5	7.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
REFUELING/MAINTENANCE, 3/7/81, SIXTEEN WEEKS
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
19	11/29/80	S	.0	H	5		RC	ZZZZZZ	LOAD DROP FOR CONTROL ROD PATTERN ADJUSTMENT

SUMMARY

PEACH BOTTOM 3 OPERATED AT OR ABOVE FULL-POWER FOR NOVEMBER, WITH 1 REDUCTION ON THE 29TH FOR CONTROL ROD SEQUENCE CHANGES.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

PILGRIM

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Massachusetts/Plymouth 4 miles SE of Plymouth
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	1998
5.	Design Electrical Rating (MWE Net)	655
6.	Date of Initial Criticality	June 16, 1972
7.	Date of Commercial Operation	December 1972
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Cape Cod Bay

Utility & Contractor Information

10.	Licensee	Boston Edison Company
11.	Corporate Address	800 Boylston Street Boston, Massachusetts 02199
12.	Corporate Contact	A. Corey, Vice President
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	GE
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	IE Section Chief	T. MARTIN
19.	IE Resident Inspector	J. JOHNSON
20.	Licensing Project Manager	M. WILLIAMS x29798
21.	Docket Number	50-293
22.	License Number/Date of Issue	DPR-35, 6/8/72
23.	Public Document Room	Plymouth Public Library North Street Plymouth, Massachusetts 02360

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

- THE PLANT WAS SHUTDOWN FOR ONE WEEK TO REPAIR SEVERAL PACKING LEAKS AND A PINHOLE LEAK IN A WELD ON THE 2 INCH LINE FROM THE REACTOR VESSEL DRAIN TO THE REACTOR WATER CLEANUP SYSTEM.
- THE NITROGEN SUPPLY TO DRYWELL INSTRUMENTATION REMAINS ISOLATED PENDING COMPLETION OF THE LICENSEE'S INVESTIGATION, CORRECTIVE ACTION AND REPORT TO THE NRC ON POST RELIEF VALVE ACTUATIONS.
- INCREASED VIBRATION WAS NOTED ON THE 'A' RECIRCULATION PUMP MOTOR GENERATOR SET GENERATOR BEARING.
- FACILITY ITEMS (PLANS AND PROCEDURES):
- INSTALLATION OF CONTINUOUS ULTRASONIC TEST EQUIPMENT ON THE SCRAM DISCHARGE VOLUME IS EXPECTED TO BE COMPLETED BY 12/1/80.

PLANT STATUS:

- AS OF 11/25/80, THE PLANT IS OPERATING AT FULL POWER WITH MAIN STACK AND OFFGAS RELEASE RATES OF ABOUT 2100 AND 52,000 MICROCURIES/SEC., RESPECTIVELY.
- LAST IE SITE INSPECTION DATE: 11/26/80
- INSPECTION REPORT NO: 50-293/80-30

SUMMARY

INSPECTION SUMMARY

- NONE

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

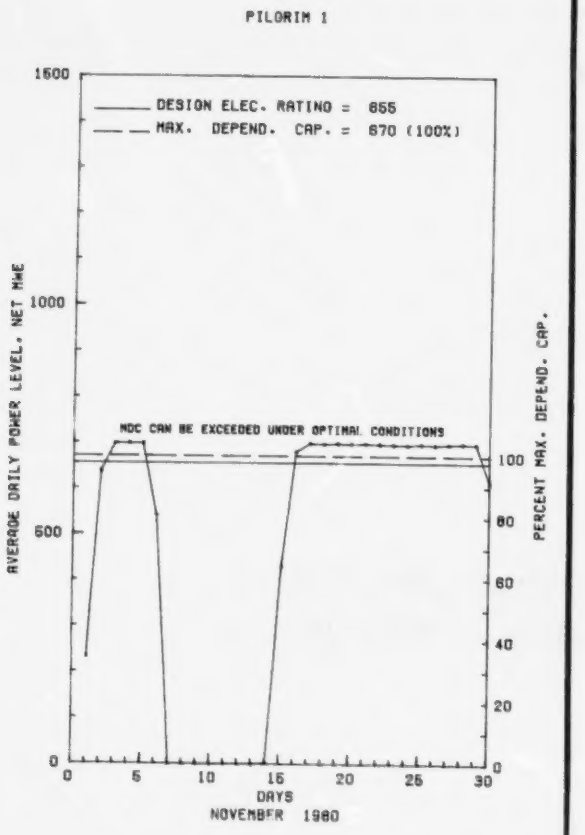
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-072/ 01T	10/02/80	10/27/80	MAIN STEAM LINE HI RAD TRIP SET POINTS WERE NOT IN ACCORDANCE WITH TECH SPECS
80-073/ 03L	09/17/80	10/17/80	MONTHLY SURVEILLANCE ON HELB PIPING INSPECTION WAS 6 DAYS LATE
80-074/ 04L	01/25/79	10/22/80	CONDENSER DELTA TEMP EXCEEDED 32 DEGREES F DURING 1979
80-075/ 03L	02/22/79	11/14/80	FIRE DETECTION EQUIPMENT INOPERABLE. (REVIEW OF PREVIOUS EVENTS AS ACTION FOLLOWING IE INSPECTION REPORT 79-20 IDENTIFIED THESE EVENTS AS REPORTABLE).
80-076/ 03L	09/23/80	10/23/80	DRYWELL FLOOR DRAIN PUMP INTEGRATOR INOP. CAUSE UNKNOWN
80-077/ 03L	09/30/80	10/30/80	CONTAINMENT ATMOSPHERIC SAMPLE SYSTEM C19 INOPERABLE DUE TO MOISTURE IN LINES
80-078/ 01T	10/30/80	11/13/80	SALT SERVICE WATER SYSTEM SEISMIC ANCHOR (SCREEN WASH PUMP) WAS IDENTIFIED TO BE OUTSIDE MISSILE PROTECTION
80-079/ 03L	10/01/80	10/31/80	'D' SAFETY-RELIEF VALVE (SRV) STUCK OPEN AFTER MANUAL OPERATION TO CONTROL RCS PRESSURE DURING SCRAM AND MSIV CLOSURE ON 10/01/80
80-080/ 01T	10/31/80	11/14/80	'A' SAFETY-RELIEF VALVE (SRV) INADVERTENTLY OPENED DUE TO HIGH NITROGEN PRESSURE
80-081/ 03L	10/05/80	11/03/80	REACTOR BLDG VENT RAD MONITOR SAMPLE PUMP FLOW ALARMS AND BLOWN FUSES DUE TO MOISTURE IN SAMPLES LINES
80-082/ 03L	10/08/80	11/07/80	CONTAINMENT ISOL VALVE MO-1001-34B WOULD NOT CLOSE DUE TO FAILED TORQUE SWITCH
80-083/ 04T	11/03/80	11/10/80	ANOMOLOUS MEASUREMENT CESIUM-137 IN CRANBERRIES GREATER THAN TEN TIMES CONTROL
80-084/ 01T	11/13/80	11/14/80	DRYWELL INSPECTION REVEALED A PINHOLE LEAK IN A COUPLING WELD IN THE 2 INCH LINE FROM R.V. DRAIN TO THE CLEANUP SYSTEM

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 1-16-81

REVIEWED BY NRR DL: M. Williams **DATE:** 1-19-81

PILGRIM

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-293
2. Utility Contact: G. G. Whitney (617) 746-7900
3. Licensed Thermal Power (MWt): 1998
4. Nameplate Rating (Gross MWe): 780 x 0.87 = 678
5. Design Electrical Rating (Net MWe): 655
6. Maximum Dependable Capacity (Gross MWe): 690
7. Maximum Dependable Capacity (Net MWe): 670
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	8,040.0	69,936.0
12. Hours Reactor Was Critical	603.8	4,459.7	49,440.4
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	515.6	4,210.4	47,762.9
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	985,296	7,719,312	80,810,424
17. Gross Electrical Energy (MWH)	342,960	2,648,970	26,813,804
18. Net Electrical Energy (MWH)	330,160	2,548,102	25,754,225
19. Unit Service Factor	71.6	52.4	68.3
20. Unit Availability Factor	71.6	52.4	68.3
21. Unit Capacity Factor (MDC Net)	68.4	47.3	55.0
22. Unit Capacity Factor (DER Net)	70.0	48.4	56.2
23. Unit Forced Outage Rate	28.4	10.0	10.6

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
11	10/31/80	F	8.7	A	4	80-80/01	SF	VALVEX	HIGH N2 PRESSURE TO SV CAUSED SRV TO LIFT. PRESSURE REDUCED.
12	11/06/80	F	195.7	A	1	80-87/03	CI	VALVEX	UNIDENTIFIED LEAKAGE IN DRYWELL. VARIOUS COMPONENTS REPAIRED.

SUMMARY

PILGRIM 1 OPERATED NORMALLY DURING NOVEMBER, WITH 2 OUTAGES (ONE CONTINUING FROM 10/31/80, AND ANOTHER ON 11/06) FOR EQUIPMENT FAILURE.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

POINT BEACH 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Wisconsin/Manitowoc 15 miles N of Manitowoc
2.	Electric Reliability Council	Mid-America Interpool Network
3.	Type of Reactor	PWR
4.	Power Level (MWT)	1518
5.	Design Electrical Rating (MWE Net)	497
6.	Date of Initial Criticality	November 2, 1970
7.	Date of Commercial Operation	December 21, 1970
8.	Condenser Cooling Method	Straight Through
9.	Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10.	Licensee	Wisconsin Electric Power Co.
11.	Corporate Address	231 West Michigan Milwaukee, Wisconsin
12.	Corporate Contact	S. Burstein
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	R. F. Warnick
19.	IE Resident Inspector	W. Guldemond
20.	Licensing Project Manager	C. M. Trammell
21.	Docket Number	50-266
22.	License Number/Date of Issue	DPR-24 issued 10/05/70
23.	Public Document Room	Joseph Mann Public Library Two Rivers, Wisconsin Manitowoc Public Library Manitowoc, Wisconsin (Temporary LPDR for spent fuel pool capacity increase amendment only

INSPECTION STATUS

ENFORCEMENT STATUS

10 CFR 20.101 LIMITS WHOLE BODY RADIATION DOSE TO 18-YEAR-OLDS TO 1.25 REM PER CALENDAR QUARTER. CONTRARY TO THE ABOVE, ONE 18-YEAR-OLD IN 1975 AND ONE IN 1979 RECEIVED WHOLE BODY DOSES EXCEEDING 1.25 REM IN A CALENDAR QUARTER.
(8016 INFRACTION)

10 CFR 71.3 REQUIRES EITHER A LICENSE OR AN EXEMPTION FOR SHIPMENTS OF TYPE B QUANTITIES OF RADIOACTIVE MATERIAL. CONTRARY TO THE ABOVE, ON JUNE 10 AND 13, 1980, TYPE B QUANTITIES OF NORMAL FORM RADIOACTIVE MATERIAL WERE SHIPPED IN MODEL NO. AL-33-90 PACKAGING WITHOUT EITHER A LICENSE FOR SUCH SHIPMENTS OR AN EXEMPTION FROM PART 71 REQUIREMENTS.
(8016 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

+ THE UNIT OPERATED ROUTINELY DURING THE MONTH AND FOLLOWING A PLANT TRIP, STARTED A REFUELING OUTAGE ON NOVEMBER 26, 1980.

\+LAST IE SITE INSPECTION DATE: SEPTEMBER 8-19, 1980

+ INSPECTION REPORT NO: 80-16

SUMMARY

INSPECTION SUMMARY

+ HEALTH PHYSICS APPRAISAL ON SEPTEMBER 8-19, (80-16): SPECIAL, ANNOUNCED APPRAISAL OF HEALTH PHYSICS PROGRAM, INCLUDING ORGANIZATION AND MANAGEMENT, TRAINING AND QUALIFICATIONS, EXPOSURE CONTROLS, RADIOLOGICAL CONTROLS, SURVEILLANCE, INSTRUMENTATION, FACILITIES AND EQUIPMENT, ALARA, AND EMERGENCY RESPONSE. THE INSPECTION INVOLVED APPROXIMATELY 300 INSPECTOR-HOURS ONSITE BY FOUR NRC INSPECTORS. NO SIGNIFICANT WEAKNESSES WERE IDENTIFIED IN THE HEALTH PHYSICS PROGRAM. TWO APPARENT ITEMS OF NONCOMPLIANCE WERE IDENTIFIED (TWO INDIVIDUALS RECEIVED DOSES EXCEEDING 10 CFR 20.101 LIMITS AND IMPROPER USE OF A RADWASTE SHIPPING PACKAGE).

REPORTS RECEIVED FROM LICENSEE

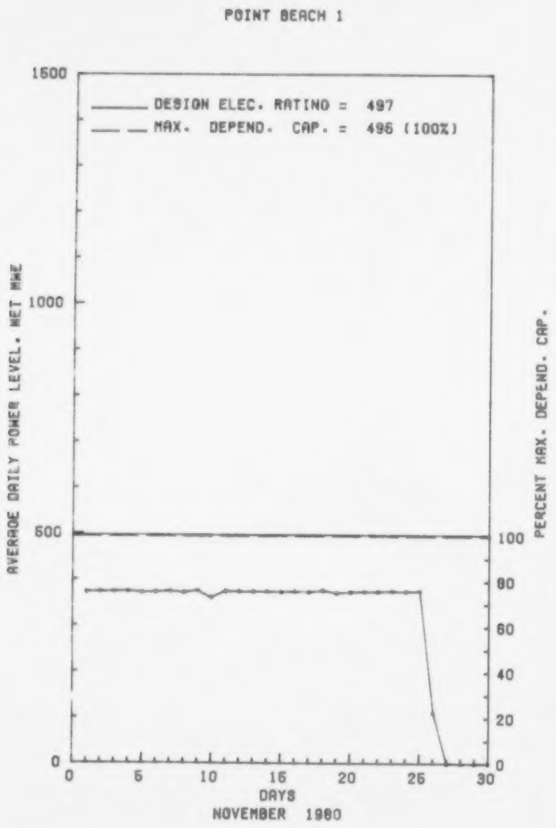
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-12/03L-0	10/05/80	10/31/80	LEAK ON A COMPRESSION FITTING ON THE INLET TO VALVE
80-13/01T-0	11/13/80	11/21/80	SUCTION TO THE 1P4B BORIC ACID TRANSFER PUMP WAS ISOLATED.

REVIEWED BY OMPA LOEB: **DATE:** 1/16/81

REVIEWED BY NRR DL: **DATE:** 1/19/81

POINT BEACH 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

ITEM 21 CALCULATED BY USING A WEIGHTED AVERAGE

1. Docket Number: 50-266
2. Utility Contact: C.W. FAY (414) 277-2811
3. Licensed Thermal Power (MWt): 1518
4. Nameplate Rating (Gross MWe): 582 × 0.9 = 524
5. Design Electrical Rating (Net MWe): 497
6. Maximum Dependable Capacity (Gross MWe): 515
7. Maximum Dependable Capacity (Net MWe): 495
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): 390
10. Reasons for Restrictions, If Any:
INHIBIT CORROSION OF STEAM GENERATOR TUBES

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	8,040.0	88,272.0
12. Hours Reactor Was Critical	634.6	6,860.4	73,376.9
13. Reactor Reserve Shutdown Hours	.4	19.9	602.1
14. Hours Generator On-Line	633.3	6,708.5	71,102.5
15. Unit Reserve Shutdown Hours	.0	152.8	755.2
16. Gross Thermal Energy (MWH)	738,200	8,069,299	98,822,096
17. Gross Electrical Energy (MWH)	239,320	2,621,230	33,255,030
18. Net Electrical Energy (MWH)	225,715	2,475,485	31,665,663
19. Unit Service Factor	88.0	83.4	80.5
20. Unit Availability Factor	88.0	85.3	81.4
*21. Unit Capacity Factor (MDC Net)	63.3	62.2	73.6
22. Unit Capacity Factor (DER Net)	63.1	62.0	72.2
23. Unit Forced Outage Rate	5.8	.4	3.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 12/28/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
5	11/26/80	F	6.6	A	3		HH	ZZZZZZ	EXCESSIVE VIBRATIONS IN THE FEEDWATER SYSTEM CAUSED A BROKEN AIR LINE TO THE "A" MAIN FEED REGULATOR VALVE. THE CLOSURE OF THE VALVE RESULTED IN A LOW STEAM GENERATOR LEVEL AND A STEAM FLOW/FEED FLOW MISMATCH WHICH CAUSED THE UNIT TRIP. THE LINE WAS REPAIRED AND THE UNIT BROUGHT BACK TO SERVICE.
6	11/26/80	F	32.1	A	3		HH	ZZZZZZ	UNIT TRIPPED AS IN (5) ABOVE. INVESTIGATIONS ARE UNDERWAY TO DETERMINE CAUSES AT THIS TIME.
7	11/29/80	S	48.0	C	4		ZZ	ZZZZZZ	UNIT REMAINED OFF LINE FOLLOWING (6) ABOVE FOR SCHEDULED REFUELING OUTAGE. UNIT SCHEDULED TO GO BACK ON LINE DECEMBER 28, 1980.

SUMMARY

UNIT 1 OPERATED AT 375 MWE NET THROUGHOUT THE PERIOD WITH ONE LOAD REDUCTION, TWO UNIT TRIPS AND A SHUTDOWN FOR THE ANNUAL REFUELING OUTAGE. LOAD WAS REDUCED TO 297 MWE NET FOR 3 HOURS ON NOVEMBER 10, 1980, TO PERFORM TURBINE STOP VALVE TESTING.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

POINT BEACH 2

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Wisconsin/Manitowoc 15 miles N of Manitowoc
2.	Electric Reliability Council	Mid-America Interpool Network
3.	Type of Reactor	PWR
4.	Power Level (MWT)	1518
5.	Design Electrical Rating (MWE Net)	497
6.	Date of Initial Criticality	May 30, 1972
7.	Date of Commercial Operation	October 1, 1972
8.	Condenser Cooling Method	Straight Through
9.	Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10.	Licensee	Wisconsin Electric Power Co.
11.	Corporate Address	231 West Michigan Milwaukee, Wisconsin
12.	Corporate Contact	S. Burstein
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	R. F. Warnick
19.	IE Resident Inspector	W. Guldemond
20.	Licensing Project Manager	C. M. Trammell
21.	Docket Number	50-301
22.	License Number/Date of Issue	DPR-27 issued 11/16/71
23.	Public Document Room	Joseph Mann Public Library Two Rivers, Wisconsin Manitowoc Public Library Manitowoc, Wisconsin (Temporary LPDR for spent fuel pool capacity increase amendment only

INSPECTION STATUS

ENFORCEMENT STATUS

10 CFR 20.101 LIMITS WHOLE BODY RADIATION DOSE TO 18-YEAR-OLDS TO 1.25 REM PER CALENDAR QUARTER. CONTRARY TO THE ABOVE, ONE 18-YEAR-OLD IN 1975 AND ONE IN 1979 RECEIVED WHOLE BODY DOSES EXCEEDING 1.25 REM IN A CALENDAR QUARTER.
(8016 INFRACTION)

10 CFR 71.3 REQUIRES EITHER A LICENSE OR AN EXEMPTION FOR SHIPMENTS OF TYPE B QUANTITIES OF RADIOACTIVE MATERIAL. CONTRARY TO THE ABOVE, ON JUNE 10 AND 13, 1980, TYPE B QUANTITIES OF NORMAL FORM RADIOACTIVE MATERIAL WERE SHIPPED IN MODEL NO. AL-33-20 PACKAGING WITHOUT EITHER A LICENSE FOR SUCH SHIPMENTS OR AN EXEMPTION FROM PART 71 REQUIREMENTS.
(8016 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

THE UNIT OPERATED ROUTINELY DURING THE MONTH.

+LAST IE SITE INSPECTION DATE: SEPTEMBER 8-19, 1980

+ INSPECTION REPORT NO: 80-16

SUMMARY

INSPECTION SUMMARY

+ HEALTH PHYSICS APPRAISAL ON SEPTEMBER 8-19, (80-16): SPECIAL, ANNOUNCED APPRAISAL OF HEALTH PHYSICS PROGRAM, INCLUDING ORGANIZATION AND MANAGEMENT, TRAINING AND QUALIFICATIONS, EXPOSURE CONTROLS, RADIOLOGICAL CONTROLS, SURVEILLANCE, INSTRUMENTATION, FACILITIES AND EQUIPMENT, ALARA, AND EMERGENCY RESPONSE. THE INSPECTION INVOLVED APPROXIMATELY 300 INSPECTOR-HOURS ONSITE BY FOUR NRC INSPECTORS. NO SIGNIFICANT WEAKNESSES WERE IDENTIFIED IN THE HEALTH PHYSICS PROGRAM. TWO APPARENT ITEMS OF NONCOMPLIANCE WERE IDENTIFIED (TWO INDIVIDUALS RECEIVED DOSES EXCEEDING 10 CFR 20.101 LIMITS AND IMPROPER USE OF A RADWASTE SHIPPING PACKAGE).

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

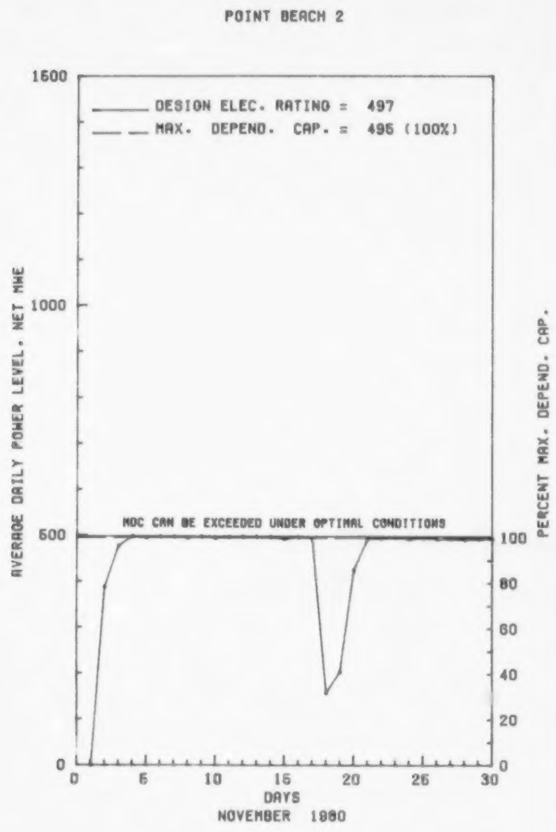
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE			

REVIEWED BY OMPA LOEB: PEBobe **DATE:** 1/16/81

REVIEWED BY NRR DL: T Colburn **DATE:** 1/9/81

POINT BEACH 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-301
2. Utility Contact: C.W. FAY (414) 277-2811
3. Licensed Thermal Power (MWt): 1518
4. Nameplate Rating (Gross MWe): 582 x 0.9 = 524
5. Design Electrical Rating (Net MWe): 497
6. Maximum Dependable Capacity (Gross MWe): 519
7. Maximum Dependable Capacity (Net MWe): 495
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

ITEM 21 CALCULATED BY USING A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	8,040.0	73,057.0
12. Hours Reactor Was Critical	698.0	6,909.1	65,806.0
13. Reactor Reserve Shutdown Hours	.6	20.2	186.3
14. Hours Generator On-Line	684.0	6,827.5	64,554.5
15. Unit Reserve Shutdown Hours	2.2	17.5	123.9
16. Gross Thermal Energy (MWH)	989,299	10,022,778	87,675,314
17. Gross Electrical Energy (MWH)	338,720	3,384,100	29,745,410
18. Net Electrical Energy (MWH)	323,864	3,225,677	28,300,806
19. Unit Service Factor	95.0	84.9	88.4
20. Unit Availability Factor	95.3	85.1	88.5
*21. Unit Capacity Factor (MDC Net)	90.9	81.1	79.0
22. Unit Capacity Factor (DER Net)	90.5	80.7	77.9
23. Unit Forced Outage Rate	1.5	5.2	1.9

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUELING - APRIL 17, 1981 5 WEEKS
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
6	11/01/80	S	25.9	B	1		HB	XXXXXX	UNIT 2 WAS SHUT DOWN TO PERFORM NUMEROUS SECONDARY SYSTEM REPAIRS TO ENSURE CONTINUED OPERATION DURING THE UPCOMING UNIT 1 REFUELING.
7	11/18/80	F	10.1	A	1	80-010	SF	PUMPXX	UNIT WAS SHUT DOWN TO MEET TECH. SPEC. REQUIREMENTS FOR HAVING ONE SAFETY INJECTION PUMP OUT-OF-SERVICE FOR A PERIOD GREATER THAN 24 HOURS. SAFETY INJECTION PUMP, 2P15A, WAS FOUND TO HAVE BROKEN KEYS HOLDING THE COUPLING ON BOTH THE MOTOR AND PUMP. PUMP WAS REPAIRED AND UNIT WAS BROUGHT BACK TO SERVICE.
8	11/19/80	F	.8	H	4		ZZ	ZZZZZZ	UNIT WAS HELD AT 50% POWER DUE TO DELTA FLUX CONSIDERATION FOLLOWING THE STARTUP FROM SAFETY INJECTION PUMP OUTAGE.

SUMMARY

UNIT 2 OPERATED AT 495 MWE NET THROUGHOUT THE PERIOD WITH TWO LOAD REDUCTIONS AND TWO SHUT-DOWNS. LOAD WAS REDUCED TO 420 MWE NET FOR FIVE HOURS ON NOV. 3, AT THE REQUEST OF POWER SUPPLY, AND TO 435 MWE FOR APPROXIMATELY TWO HOURS ON NOV. 15, AGAIN, AT THE REQUEST OF POWER SUPPLY.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

PRAIRIE ISLAND 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Minnesota/Goodhue 28 miles SE of Minneapolis
2.	Electric Reliability Council	Mid-Continent Area Reliability Coordination Agreement
3.	Type of Reactor	PWR
4.	Power Level (MWT)	1650
5.	Design Electrical Rating (MWE Net)	530
6.	Date of Initial Criticality	December 1, 1973
7.	Date of Commercial Operation	December 16, 1973
8.	Condenser Cooling Method	Cooling Towers
9.	Condenser Cooling Water Source	Mississippi River

Utility & Contractor Information

10.	Licensee	Northern States Power Company
11.	Corporate Address	414 Nicollet Mall - 8th Floor Minneapolis, Minnesota 55401
12.	Corporate Contact	G. H. Neils (Oper.) L. O. Mayer (Lic.)
13.	Architect/Engineer	Pioneer Services & Eng. Co.
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Northern States Power Co.

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	W. S. Little
19.	IE Resident Inspector	C. D. Feierabend 612-388-8209
20.	Licensing Project Manager	R. E. Martin
21.	Docket Number	50-282
22.	License Number/Date of Issue	DPR-42 issued 8/9/73
23.	Public Document Room	Environmental Conservation Library Minneapolis Public Library 300 Nicollet Mall Minneapolis, Minnesota 55401

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

OPERATING NORMALLY.

♦LAST IE SITE INSPECTION DATE: OCTOBER 8-9, 1980

♦ INSPECTION REPORT NO: 80-16

SUMMARY

INSPECTION SUMMARY

♦ INSPECTION ON SEPTEMBER 1-30, (80-15): ROUTINE RESIDENT INSPECTION OF PLANT OPERATION, MAINTENANCE, SURVEILLANCE, SECURITY, RADIATION PROTECTION, REFUELING ACTIVITIES, FOLLOWUP ON IE CIRCULARS AND FOLLOWUP ON PREVIOUS INSPECTION FINDINGS. THE INSPECTION INVOLVED 133 INSPECTOR-HOURS ONSITE BY TWO NRC INSPECTORS INCLUDING 33 INSPECTOR-HOURS ONSITE DURING OFF-SHIFTS. NO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED.

♦ INSPECTION ON OCTOBER 8-9, (80-16): ROUTINE, ANNOUNCED INSPECTION OF CONTAINMENT INTEGRATED LEAK RATE TEST. THE INSPECTION INVOLVED 21 INSPECTOR-HOURS ONSITE BY ONE NRC INSPECTOR INCLUDING 6 INSPECTOR-HOURS ONSITE DURING OFF-SHIFTS. NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

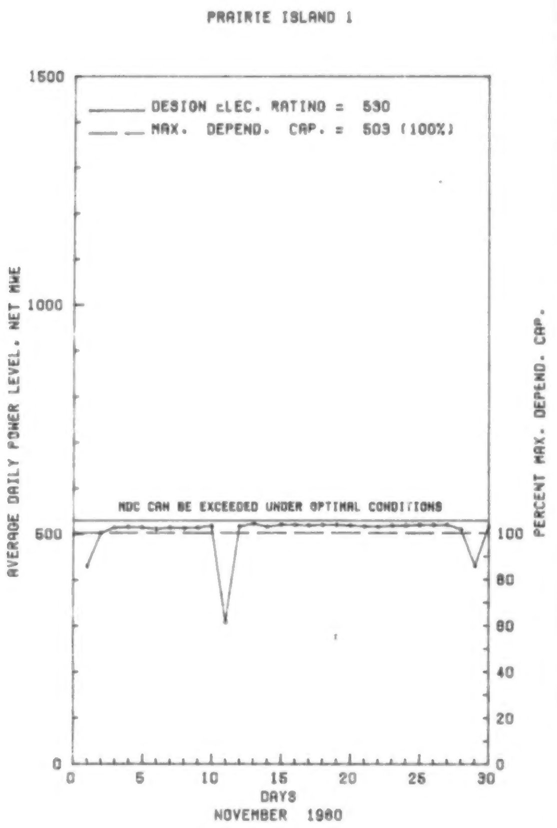
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-27/ 03L-0	09/19/80	10/17/80	STEAM FLOW TRANSMITTER DRIFT
80-31/ 03L-0	10/17/80	11/14/80	MISSED SP 1112, STEAM EXCLUSION TEST.
80-34/ 03L-0	10/31/80	11/24/80	INOPERABILITY OF NO. 122 AUXILIARY BUILDING SPECIAL VENT SYSTEM.

REVIEWED BY OMPA LOEB: Shery J Massaro **DATE:** 1/16/81

REVIEWED BY NRR DL: **DATE:** 01/16/81

PRAIRIE ISLAND 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-282
2. Utility Contact: A. A. Hunstad (612) 388-4767
3. Licensed Thermal Power (MWt): 1650
4. Nameplate Rating (Gross MWe): 659 x 0.9 = 593
5. Design Electrical Rating (Net MWe): 530
6. Maximum Dependable Capacity (Gross MWe): 534
7. Maximum Dependable Capacity (Net MWe): 503
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	8,040.0	61,008.0
12. Hours Reactor Was Critical	713.8	6,214.7	47,423.1
13. Reactor Reserve Shutdown Hours	6.2	6.2	5,538.1
14. Hours Generator On-Line	711.9	6,123.1	46,246.7
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,154,533	9,428,805	71,502,799
17. Gross Electrical Energy (MWH)	385,467	2,921,880	23,075,550
18. Net Electrical Energy (MWH)	362,915	2,773,801	21,563,053
19. Unit Service Factor	98.9	76.2	75.8
20. Unit Availability Factor	98.9	76.2	75.8
21. Unit Capacity Factor (MDC Net)	100.2	67.4	70.3
22. Unit Capacity Factor (DER Net)	95.1	63.9	66.7
23. Unit Forced Outage Rate	1.1	2.9	12.4

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
REFUELING, SEPTEMBER 1981, 6 WEEKS
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	11/11/80	F	8.1	G	3				UNIT TRIP AND SPURIOUS SAFETY INJECTION CAUSED BY ERROR IN PERFORMING SAFEGUARDS LOGIC TEST.

SUMMARY

THE UNIT WAS IN LOAD FOLLOW OPERATION DURING THE MONTH OF NOVEMBER, BUT STILL OPERATED AT ABOVE 100% CAPACITY. AT 928 ON 11-11 AN ERROR IN PERFORMING A SAFEGUARDS LOGIC TEST RESULTED IN A SPURIOUS SAFETY INJECTION AND UNIT TRIP. THE UNIT RETURNED TO SERVICE AT 1733 THE SAME DAY.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

PRAIRIE ISLAND 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Minnesota/Goodhue 28 miles SE of Minneapolis
2.	Electric Reliability Council	Mid-Continent Area Reliability Coordination Agreement
3.	Type of Reactor	PWR
4.	Power Level (MWT)	1650
5.	Design Electrical Rating (MWE Net)	530
6.	Date of Initial Criticality	December 17, 1974
7.	Date of Commercial Operation	December 21, 1974
8.	Condenser Cooling Method	Cooling Towers
9.	Condenser Cooling Water Source	Mississippi River

Utility & Contractor Information

10.	Licensee	Northern States Power Company
11.	Corporate Address	414 Nicollet Mall - 8th Floor Minneapolis, Minnesota 55401
12.	Corporate Contact	G. H. Neils (Oper.) L. O. Mayer (Lic.)
13.	Architect/Engineer	Pioneer Services & Eng. Co.
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Northern States Power Co.

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	W. S. Little
19.	IE Resident Inspector	C. D. Feierabend 612-388-8209
20.	Licensing Project Manager	R. Martin
21.	Docket Number	50-306
22.	License Number/Date of Issue	DPR-60 issued 10/29/74
23.	Public Document Room	Environmental Conservation Library Minneapolis Public Library 300 Nicollet Mall Minneapolis, Minnesota 55401

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

OPERATING ROUTINELY.

LAST IE SITE INSPECTION DATE: SEPTEMBER 1-30, 1980

+ INSPECTION REPORT NO: 80-16

SUMMARY

INSPECTION SUMMARY

+ INSPECTION ON SEPTEMBER 1-30, (80-16): ROUTINE RESIDENT INSPECTION OF PLANT OPERATION, MAINTENANCE, SURVEILLANCE, SECURITY, RADIATION PROTECTION, REFUELING ACTIVITIES, FOLLOWUP ON IE CIRCULARS AND FOLLOWUP ON PREVIOUS INSPECTION FINDINGS. THE INSPECTION INVOLVED 133 INSPECTOR-HOURS ONSITE BY TWO NRC INSPECTORS INCLUDING 33 INSPECTOR-HOURS ONSITE DURING OFF-SHIFTS. NO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED.

REPORTS RECEIVED FROM LICENSEE

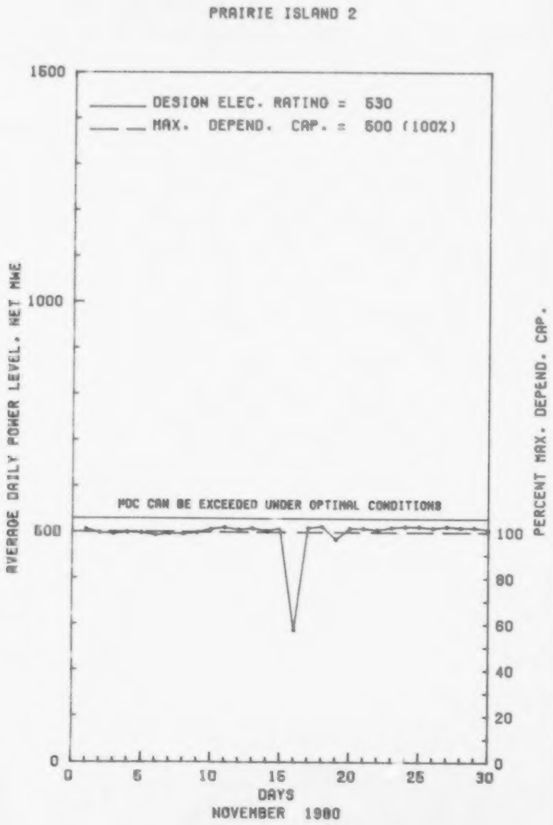
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-28/ 03L-0	10/03/80	10/31/80	CONTROL SWITCH FOR NO. 24 FAN-COIL UNIT FOUND IN MANUAL.
80-29/ 03L-0	10/04/80	10/31/80	FAILURE OF NO. 23 INVERTER.

REVIEWED BY OMPA LOEB: Sheryl Massaro **DATE:** 1/16/81

REVIEWED BY NRR DL: R Martin **DATE:** 1/16/81

PRAIRIE ISLAND 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-306
2. Utility Contact: A. A. Hunstad (612) 388-4767
3. Licensed Thermal Power (MWt): 1650
4. Nameplate Rating (Gross MWe): 659 x 0.9 = 593
5. Design Electrical Rating (Net MWe): 530
6. Maximum Dependable Capacity (Gross MWe): 531
7. Maximum Dependable Capacity (Net MWe): 500
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	8,040.0	52,126.0
12. Hours Reactor Was Critical	720.0	6,508.8	45,637.8
13. Reactor Reserve Shutdown Hours	0	102.0	1,503.0
14. Hours Generator On-Line	720.0	6,425.9	44,828.9
15. Unit Reserve Shutdown Hours	0	0	0
16. Gross Thermal Energy (MWH)	1,164,750	10,219,362	69,891,865
17. Gross Electrical Energy (MWH)	380,620	3,308,040	22,342,170
18. Net Electrical Energy (MWH)	357,979	3,095,823	20,934,455
19. Unit Service Factor	100.0	79.9	86.0
20. Unit Availability Factor	100.0	79.9	86.0
21. Unit Capacity Factor (MDC Net)	99.4	77.0	80.3
22. Unit Capacity Factor (DER Net)	93.8	72.7	75.8
23. Unit Forced Outage Rate	0	.6	4.2

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUELING, FEBRUARY 1981, 6 WEEKS
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
	11/16/80	S	0.0	B					POWER REDUCTION OF 42% FOR TURBINE VALVE TEST AND CONDENSER CLEANING.

SUMMARY

THE UNIT WAS BASE LOADED DURING THE MONTH AND ACHIEVED NEARLY A 100% CAPACITY FACTOR. THERE WERE NO SHUTDOWNS AND ONE REDUCTION FOR TURBINE VALVE TEST AND CONDENSER CLEANING.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] **Exhibit F:**
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] **Exhibit H:**
Same source.

QUAD CITIES 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Illinois/Rock Island 20 miles NE of Moline
2.	Electric Reliability Council	Mid-America Interpool Network
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2511
5.	Design Electrical Rating (MWE Net)	789
6.	Date of Initial Criticality	October 18, 1971
7.	Date of Commercial Operation	February 18, 1973
8.	Condenser Cooling Method	Spray Canal
9.	Condenser Cooling Water Source	Mississippi River

Utility & Contractor Information

10.	Licensee	Commonwealth Edison Co.
11.	Corporate Address	P. O. Box 767 Chicago, Illinois 60690
12.	Corporate Contact	Cordell Reed, Vice President
13.	Architect/Engineer	Sargent and Lundy
14.	Nuclear Steam Supply System	GE
15.	Constructor	United Engineers & Constructors, Inc.

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	R. Spessard
19.	IE Resident Inspector	N. J. Chrissotimos (309) 654-2227
20.	Licensing Project Manager	R. B. Bevan
21.	Docket Number	50-254
22.	License Number/Date of Issue	DPR-29 issued 10/1/71
23.	Public Document Room	Moline Public Library 504 17th Street Moline, Illinois 61265

INSPECTION STATUS

ENFORCEMENT STATUS

TECHNICAL SPECIFICATION 6.2.B REQUIRES ADHERENCE TO STATION RADIATION CONTROL PROCEDURES. THE FOLLOWING INSTANCES OF FAILURE TO MEET THIS REQUIREMENT WERE IDENTIFIED DURING THE APPRAISAL. (A) STATION PROCEDURES QRS700-2 AND QRS700-3, RESPECTIVELY, REQUIRE QUARTERLY CALIBRATION OF "CP" AND GEIGER-MUELLER (GM) EXPOSURE RATE INSTRUMENTS. CONTRARY TO THE ABOVE, ON MAY 12, 1980, VICTOREEN 740F (CP) S/N 109 AND EBERLINE PRM 4 (GM) S/N 2216 LOCATED IN EMERGENCY BOX NO. 1 WERE FOUND TO HAVE BEEN LAST CALIBRATED ON OCTOBER 26, 1979, APPROXIMATELY SIX MONTHS EARLIER. (B) STATION PROCEDURE QRP 100-1, SECTION 26, SPECIFIES A LIMIT OF 3000 COUNTS PER MINUTES ON LAUNDERED PROTECTIVE CLOTHING MADE AVAILABLE FOR REUSE. CONTRARY TO THE ABOVE, SEVERAL GARMENTS TAKEN FROM REISSUE BINS AND MONITORED WITH THE LAUNDRY MONITOR DURING THE APPRAISAL EXCEEDED THIS LIMIT.
(8020 INFRACTION)

UNIT 1 TECHNICAL SPECIFICATION 3.10.B.2 STATES, "THE SRM OR DUNKING TYPE DETECTOR SHALL HAVE A MINIMUM OF 3 CPS WITH ALL RODS FULLY INSERTED IN THE CORE EXCEPT WHEN BOTH OF THE FOLLOWING CONDITIONS ARE FULFILLED: (A) NO MORE THAN TWO FUEL ASSEMBLIES ARE PRESENT IN THE CORE QUADRANT ASSOCIATED WITH THE SRM; (B) WHILE IN CORE, THESE FUEL ASSEMBLIES ARE IN LOCATIONS ADJACENT TO THE SRM." CONTRARY TO THE ABOVE, ON SEPTEMBER 14, 1980, A FUEL ASSEMBLY WAS REMOVED FROM A CORE QUADRANT CONTAINING 3 FUEL ASSEMBLIES WITH A SRM INDICATING LESS THAN 3 CPS.
(8022 INFRACTION)

UNITS 1 AND 2 TECHNICAL SPECIFICATIONS 3.12.F.2 STATES, "WITH ONE OR MORE PENETRATION FIRE BARRIERS NOT INTACT, ESTABLISH A CONTINUOUS FIRE WATCH ON AT LEAST ONE SIDE OF THE AFFECTED PENETRATION WITHIN ONE HOUR IF THE AREA ON EITHER SIDE OF THE AFFECTED PENETRATION CONTAINS EQUIPMENT REQUIRED TO BE OPERABLE." CONTRARY TO THE ABOVE, ON SEPTEMBER 10, 1980, THE NRC INSPECTOR FOUND THAT A CONTINUOUS FIRE WATCH HAD NOT BEEN ESTABLISHED WITHIN ONE HOUR ON EITHER SIDE OF TWO (2) FOUR AND ONE-HALF INCH DIAMETER FIRE BARRIER PENETRATIONS NOT SEALED IN THE FLOOR BETWEEN THE CABLE SPREADING ROOM AND THE AUXILIARY ELECTRIC ROOM.
(8022 INFRACTION)

UNIT 1 TECHNICAL SPECIFICATION 6.2.A.2 STATES, IN PART, THAT "DETAILED WRITTEN PROCEDURES INCLUDING APPLICABLE CHECKOFF LISTS SHALL BE PREPARED, APPROVED AND ADHERED TO FOR REFUELING OPERATIONS." CONTRARY TO THE ABOVE, ON SEPTEMBER 19, 1980, TEMPORARY PROCEDURE 1377 WAS NOT ADHERED TO IN THAT CHARGING WATER ISOLATION VALVES WERE NOT CLOSED AS REQUIRED BY THE PROCEDURE.
(8022 INFRACTION)

UNIT 1 TECHNICAL SPECIFICATIONS, SECTION 6.2.A.6 STATES IN PART THAT DETAILED WRITTEN PROCEDURES AND APPLICABLE CHECKOFF LISTS SHALL BE PREPARED, APPROVED AND ADHERED TO FOR PREVENTIVE AND CORRECTIVE MAINTENANCE OPERATIONS WHICH COULD HAVE AN EFFECT ON THE SAFETY OF THE FACILITY. CONTRARY TO THE ABOVE, ON JUNE 21, 1980, IMPROPER ADJUSTMENTS WERE PERFORMED ON THE EXHAUST RESTRICTOR OF A MAIN STEAM ISOLATION VALVE WHICH RESULTED IN A FAILURE OF THE MSIV TO CLOSE DURING A SURVEILLANCE TEST ON JULY 3, 1980. THE LICENSEE'S CORRECTIVE AND PREVENTIVE ACTIONS HAVE BEEN REVIEWED AND APPEAR TO BE ADEQUATE; THEREFORE, NO FURTHER RESPONSE TO THIS ITEM IS REQUIRED.
(8019 DEFICIENCY)

UNIT 1 TECHNICAL SPECIFICATIONS, SECTION 6.2.A.1 REQUIRES THAT OPERATION OF THE REACTOR AND OTHER SYSTEMS AND COMPONENTS INVOLVING NUCLEAR SAFETY OF THE FACILITY BE PERFORMED IN ACCORDANCE WITH DETAILED WRITTEN PROCEDURES AND APPLICABLE CHECKOFF LISTS. CONTRARY TO THE ABOVE, ON AUGUST 17, 1980, WHILE PERFORMING SURVEILLANCE QIS-130-2 "RCIC PUMP OPERABILITY TEST" THE OPERATOR FAILED TO RETURN THE RCIC TO ITS NORMAL OPERATING CONFIGURATION AS STATED IN STEP 8.
(8019 INFRACTION)

UNIT 1 TECHNICAL SPECIFICATIONS, SECTION 6.2.A.1 REQUIRES THAT OPERATION OF THE REACTOR AND OTHER SYSTEMS AND COMPONENTS INVOLVING NUCLEAR SAFETY OF THE FACILITY BE PERFORMED IN ACCORDANCE WITH DETAILED WRITTEN PROCEDURES AND APPLICABLE CHECKOFF LISTS. CONTRARY TO THE ABOVE, DURING THE PERIOD OF AUGUST 17 TO AUGUST 18, 1980, THE RCIC FLOW CONTROLLER WAS NOT IN ITS NORMAL OPERATING CONFIGURATION AS STATED IN STEP C-5 OF THE PROCEDURE QOS-005-2 "NRMAL CONTROL ROOM INSPECTION AND SHIFT TURNOVER P ANEL CHECKS."
(8019 INFRACTION)

10 CFR 20.203(C)(2) LISTS THREE ALTERNATIVE REQUIREMENTS FOR CONTROL OF ACCESS TO HIGH RADIATION AREAS. CONTRARY TO THESE REQUIREMENTS: (A) AN UNPOSTED, UNBARRICADED, AND UNOBSERVED HIGH RADIATION AREA EXISTED NEAR THE SAMPLE HOOD ON THE 6[illegible] FOOT LEVEL OF THE UNIT 2 REACTOR BUILDING FOR SEVERAL DAYS (MAY 5-14) DURING THE APPRAISAL; AND (B) ON MAY 5, 1980, A LADDER TEMPORARILY LASHED IN PLACE GAVE READY ACCESS TO A POSTED HIGH RADIATION AREA ABOVE THE UNIT 2 CRD ACCUMULATORS.
(8020 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENTS

- WHILE TESTING ELECTROMATIC RELIEF VALVES, DURING UNIT 1 SHUTDOWN ON AUGUST 31, 1980, ONE VALVE STUCK OPEN. THE CAUSE WAS LATER DETERMINED TO BE A FAULTY SEAT ON THE PILOT VALVE. DURING U.T. TESTING OF THE "A" LOW PRESSURE SECTION OF THE UNIT 1 TURBINE AN INDICATION WAS DISCOVERED ON ONE OF THE HUBS. AFTER NRC CONSULTANTS WITNESSED THE TESTING THE DEFINITION OF THE INDICATION WAS INCONCLUSIVE. BASED ON THESE UNCERTANTIES THE LICENSEE WILL INSPECT THE "B" TURBINE SECTION TO ENSURE NO OTHER INDICATIONS EXIST. (80-22)

PLANT STATUS

THE UNIT WAS SHUT DOWN ON AUGUST 31, 1980 TO BEGIN A SCHEDULED 12 WEEK REFUELING OUTAGE.

*LAST IE SITE INSPECTION DATE: OCTOBER 31, 1980

- INSPECTION REPORT NO: 80-26

SUMMARY

INSPECTION SUMMARY

- INSPECTION ON SEPTEMBER 1 THROUGH OCTOBER 17, (80-22): ROUTINE, UNANNOUNCED RESIDENT INSPECTION OF LICENSEE ACTION ON PREVIOUS INSPECTION FINDINGS, MAINTENANCE OBSERVATION, SURVEILLANCE OBSERVATION, OPERATIONAL SAFETY VERIFICATION, RECEIPT OF NEW FUEL, PREPARATION FOR REFUELING, REFUELING ACTIVITIES, PARTICIPATION IN NRR/LICENSEE MEETING, REVIEW OF PERIODIC AND SPECIAL REPORTS, LICENSEE EVENT REPORT FOLLOWUP, IE BULLETINS, FOLLOWUP ON HEADQUARTERS REQUEST, LOCAL LEAK RATE TESTING, INDEPENDENT INSPECTION, PLANT SCRAMS, AND ALLEGATION FOLLOWUP. THE INSPECTION INVOLVED 282 INSPECTOR-HOURS ONSITE BY TWO NRC INSPECTORS INCLUDING 58 INSPECTOR-HOURS ONSITE DURING OFF-SHIFTS. OF THE 16 AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED IN 14 AREAS; THREE ITEMS OF NONCOMPLIANCE WERE IDENTIFIED IN TWO AREAS (FAILURE TO FOLLOW PROCEDURES; AND FAILURE TO FOLLOW PROCEDURES; AND INADEQUATE MANAGEMENT).
- MANAGEMENT MEETING ON OCTOBER 31, (80-26): MANAGEMENT MEETING HELD AT THE NRC'S REQUEST TO DISCUSS THE REGULATORY PERFORMANCE OF THE ACTIVITIES AT DRESDEN, QUAD-CITIES, AND ZION NUCLEAR GENERATING STATIONS AS CONCLUDED IN THE SYSTEMATIC ASSESSMENT OF LICENSEE PERFORMANCE (SALP) PROGRAM. THE MEETING INVOLVED 15 MAN-HOURS AT THE CORPORATE OFFICES BY 11 NRC PERSONNEL. A SUMMATION OF THE LICENSEE PERFORMANCE EVALUATION WAS PRESENTED. AREAS OF CONCERN WERE DISCUSSED WITH CORPORATE MANAGEMENT. THE PERFORMANCE AT DRESDEN, QUAD-CITIES AND ZION NUCLEAR GENERATING STATIONS WAS CONSIDERED TO BE ADEQUATE.

REPORTS RECEIVED FROM LICENSEE

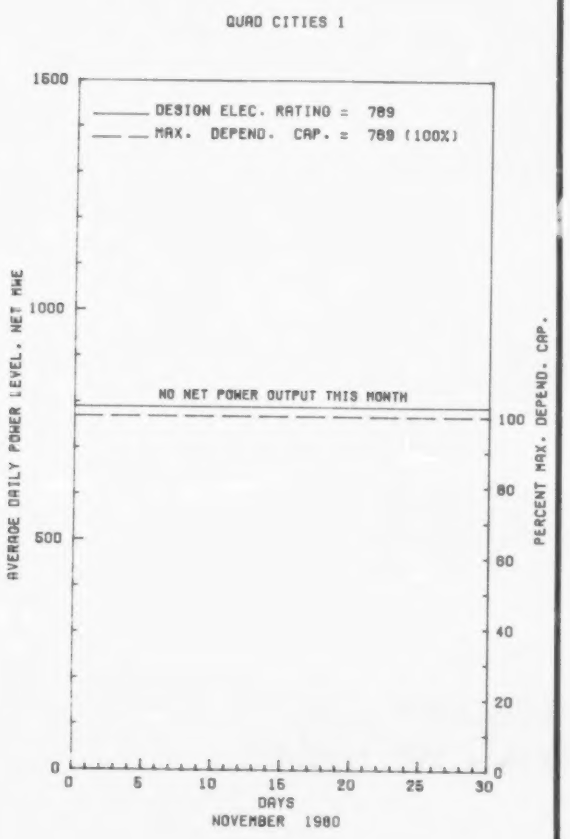
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-26/03L-0	10/11/80	10/29/80	MOTOR SHORT IN DIESEL GENERATOR COOLING PUMP RESULTED IN D-G FAILURE.
80-27/01T-0	10/15/80	10/29/80	PIPING IDENTIFIED WHICH DOES NOT MEET OPERABILITY CRITERIA PER IEB 79-14.

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 1-16-81

REVIEWED BY NRR DL: RB Bevan **DATE:** 1/19/81

QUAD CITIES 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-254
2. Utility Contact: R. TUBBS (309) 654-2241 X176
3. Licensed Thermal Power (MWt): 2511
4. Nameplate Rating (Gross MWe): 920 x 0.9 = 828
5. Design Electrical Rating (Net MWe): 789
6. Maximum Dependable Capacity (Gross MWe): 811
7. Maximum Dependable Capacity (Net MWe): 769
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	8,040.0	75,050.0
12. Hours Reactor Was Critical	0	5,725.1	60,539.2
13. Reactor Reserve Shutdown Hours	0	0	3,421.9
14. Hours Generator On-Line	0	5,621.1	57,665.0
15. Unit Reserve Shutdown Hours	0	0	909.2
16. Gross Thermal Energy (MWH)	0	11,396,718	135,938,659
17. Gross Electrical Energy (MWH)	35	3,662,846	32,765,503
18. Net Electrical Energy (MWH)	-2,652	3,358,261	30,773,777
19. Unit Service Factor	0	69.9	76.9
20. Unit Availability Factor	0	69.9	78.1
21. Unit Capacity Factor (MDC Net)	0	54.3	60.3
22. Unit Capacity Factor (DER Net)	0	52.9	58.8
23. Unit Forced Outage Rate	0	3.6	7.4

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 12/29/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
13	08/31/80	S	720.0	C	4		RC	FUELXX	CONTINUATION OF END OF CYCLE FIVE REFUELING OUTAGE.

SUMMARY

QUAD CITIES 1 REMAINED SHUT DOWN FOR NOVEMBER IN A CONTINUING REFUELING OUTAGE.

LEGEND

[1] Type:
F-Forced
S-Scheduled

[2] Reason:
A-Equipment Failure
B-Maintenance or Test
C-Refueling
D-Regulatory Restriction
E-Operator Training & License Examination
F-Administrative
G-Operational Error
H-Other

[3] Method:
1-Manual
2-Manual Scram
3-Automatic Scram
4-Continuations
5-Load Reduction
9-Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

QUAD CITIES 2

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Illinois/Rock Island 20 miles NE of Moline
2.	Electric Reliability Council	Mid-America Interpool Network
3.	Type of Reactor	BWR
4.	Power Level (MWT)	2511
5.	Design Electrical Rating (MWE Net)	789
6.	Date of Initial Criticality	April 26, 1972
7.	Date of Commercial Operation	March 10, 1973
8.	Condenser Cooling Method	Spray Canal
9.	Condenser Cooling Water Source	Mississippi River

Utility & Contractor Information

10.	Licensee	Commonwealth Edison Co.
11.	Corporate Address	P. O. Box 767 Chicago, Illinois 60690
12.	Corporate Contact	Cordell Reed, Vice President
13.	Architect/Engineer	Sargent & Lundy
14.	Nuclear Steam Supply System	GE
15.	Constructor	United Engineers & Constructors, Inc.

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	R. Spessard
19.	IE Resident Inspector	N. J. Chrissotimos (309) 654-2227
20.	Licensing Project Manager	R. B. Bevan
21.	Docket Number	50-265
22.	License Number/Date of Issue	DPR-30 issued 3/21/72
23.	Public Document Room	Moline Public Library 504 17th Street Moline, Illinois 61265

INSPECTION STATUS

ENFORCEMENT STATUS

10 CFR 20.203(C)(2) LISTS THREE ALTERNATIVE REQUIREMENTS FOR CONTROL OF ACCESS TO HIGH RADIATION AREAS. CONTRARY TO THESE REQUIREMENTS: (A) AN UNPOSTED, UNBARRICADED, AND UNOBSERVED HIGH RADIATION AREA EXISTED NEAR THE SAMPLE HOOD ON THE 647 FOOT LEVEL OF THE UNIT 2 REACTOR BUILDING FOR SEVERAL DAYS (MAY 5-14) DURING THE APPRAISAL; AND (B) ON MAY 5, 1980, A LADDER TEMPORARILY LASHED IN PLACE GAVE READY ACCESS TO A POSTED HIGH RADIATION AREA ABOVE THE UNIT 2 CRD ACCUMULATORS.
(8022 INFRACTION)

TECHNICAL SPECIFICATION 6.2.B REQUIRES ADHERENCE TO STATION RADIATION CONTROL PROCEDURES. THE FOLLOWING INSTANCES OF FAILURE TO MEET THIS REQUIREMENT WERE IDENTIFIED DURING THE APPRAISAL. (A) STATION PROCEDURES QRS700-2 AND QRS700-3, RESPECTIVELY, REQUIRE QUARTERLY CALIBRATION OF "CP" AND GEIGER-MUELLER (GM) EXPOSURE RATE INSTRUMENTS. CONTRARY TO THE ABOVE, ON MAY 12, 1980, VICTOREEN 740F (CP) S/N 109 AND EBERLINE PRM 4 (GM) S/N 2216 LOCATED IN EMERGENCY BOX NO. 1 WERE FOUND TO HAVE BEEN LAST CALIBRATED ON OCTOBER 26, 1979, APPROXIMATELY SIX MONTHS EARLIER. (B) STATION PROCEDURE QRP 100-1, SECTION 26, SPECIFIES A LIMIT OF 3000 COUNTS PER MINUTES ON LAUNDERED PROTECTIVE CLOTHING MADE AVAILABLE FOR REUSE. CONTRARY TO THE ABOVE, SEVERAL GARMENTS TAKEN FROM REISSUE BINS AND MONITORED WITH THE LAUNDRY MONITOR DURING THE APPRAISAL EXCEEDED THIS LIMIT.
(8022 INFRACTION)

UNITS 1 AND 2 TECHNICAL SPECIFICATIONS 3.12.F.2 STATES, "WITH ONE OR MORE PENETRATION FIRE BARRIERS NOT INTACT, ESTABLISH A CONTINUOUS FIRE WATCH ON AT LEAST ONE SIDE OF THE AFFECTED PENETRATION WITHIN ONE HOUR IF THE AREA ON EITHER SIDE OF THE AFFECTED PENETRATION CONTAINS EQUIPMENT REQUIRED TO BE OPERABLE." CONTRARY TO THE ABOVE, ON SEPTEMBER 10, 1980, THE NRC INSPECTOR FOUND THAT A CONTINUOUS FIRE WATCH HAD NOT BEEN ESTABLISHED WITHIN ONE HOUR ON EITHER SIDE OF TWO (2) FOUR AND ONE-HALF INCH DIAMETER FIRE BARRIER PENETRATIONS NOT SEALED IN THE FLOOR BETWEEN THE CABLE SPREADING ROOM AND THE AUXILIARY ELECTRIC ROOM.
(8024 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENTS

+ DURING UNIT 2 SHUTDOWN ON OCTOBER 17, 1980, WHILE TESTING ELECTROMATIC RELIEF VALVES ONE VALVE FAILED TO OPEN. INITIAL CAUSE IS BELIEVED TO BE EXCESSIVE LEAKAGE PAST A VALVE PLUG, WHICH PREVENTS THE DEVELOPMENT OF SUFFICIENT DIFFERENTIAL PRESSURE TO OPEN THE VALVE. UNIT 2 SHUTDOWN ON OCTOBER 17, 1980 FOR A THREE-DAY MAINTENANCE OUTAGE WHICH INCLUDED BATTERY DISCHARGE TESTING AND VARIOUS TMI MODIFICATIONS WHICH REQUIRE A DUAL UNIT OUTAGE FOR IMPLEMENTATION. (80-24)

PLANT STATUS

PLANT OPERATING ROUTINELY.

+LAST IE SITE INSPECTION DATE: OCTOBER 31, 1980

+ INSPECTION REPORT NO: 80-27

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-22/ 03L-0	10/05/80	10/15/80	ONE MAIN STEAM ISOLATION VALVE FOUND TO CLOSE FASTER THAN TS LIMIT OF 3 SECONDS.
80-23/ 03L-0	10/10/80	11/06/80	WHILE PERFORMING LPCI MOTOR OPERATED VALVE OPERABILITY TEST, MO-2-1001-29A VALVE INITIALLY FAILED TO OPEN FULLY PER TS REQUIREMENTS.
80-24/ 03L-0	10/10/80	10/23/80	LIMIT SWITCH MALADJUSTMENT CAUSED FAILURE OF CONT. COOLING WATER VALVE TO OPEN.
80-30/ 03L-0	10/31/80	11/20/80	AFTER VENTING PRIMARY CONTAINMENT VALVE AO-2-1601-63, STANDBY GAS SUCTION VENT, REQUIRED ABOUT 40 SECONDS TO CLOSE AS COMPARED TO THE TS LIMIT OF 10 SECONDS (MAXIMUM).
80-31/ 01T-0	11/16/80	11/24/80	AN ORDERLY SHUTDOWN WAS INITIATED PER TS REQUIREMENTS FOLLOWING A DETERMINATION THAT RCIC AND HPCI WERE INOPERABLE.

===

SUMMARY

INSPECTION SUMMARY

+ INSPECTION ON SEPTEMBER 1 THROUGH OCTOBER 17, (80-24): ROUTINE, UNANNOUNCED RESIDENT INSPECTION OF LICENSEE ACTION ON PREVIOUS INSPECTION FINDINGS, MAINTENANCE OBSERVATION, SURVEILLANCE OBSERVATION, OPERATIONAL SAFETY VERIFICATION, RECEIPT OF NEW FUEL, PREPARATION FOR REFUELING, REFUELING ACTIVITIES, PARTICIPATION IN NRR/LICENSEE MEETING, REVIEW OF PERIODIC AND SPECIAL REPORTS, LICENSEE EVENT REPORT FOLLOWUP, IE BULLETINS, FOLLOWUP ON HEADQUARTERS REQUEST, LOCAL LEAK RATE TESTING, INDEPENDENT INSPECTION, PLANT SCRAMS, AND ALLEGATION FOLLOWUP. THE INSPECTION INVOLVED 282 INSPECTOR-HOURS ONSITE BY TWO NRC INSPECTORS INCLUDING 58 INSPECTOR-HOURS ONSITE DURING OFF-SHIFTS. OF THE 16 AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE WERE IDENTIFIED IN 14 AREAS; THREE ITEMS OF NONCOMPLIANCE WERE IDENTIFIED IN TWO AREAS (FAILURE TO FOLLOW PROCEDURES; AND FAILURE TO FOLLOW PROCEDURES; AND INADEQUATE MANAGEMENT).

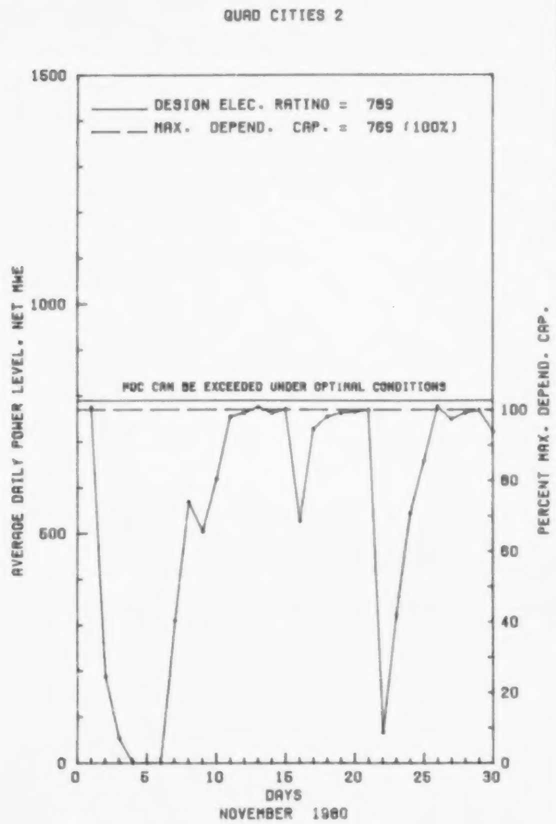
+ MANAGEMENT MEETING ON OCTOBER 31, (80-27): MANAGEMENT MEETING HELD AT THE NRC'S REQUEST TO DISCUSS THE REGULATORY PERFORMANCE OF THE ACTIVITIES AT DRESDEN, QUAD-CITIES, AND ZION NUCLEAR GENERATING STATIONS AS CONCLUDED IN THE SYSTEMATIC ASSESSMENT OF LICENSEE PERFORMANCE (SALP) PROGRAM. THE MEETING INVOLVED 18 MAN-HOURS AT THE CORPORATE OFFICES BY 11 NRC PERSONNEL. A SUMMATION OF THE LICENSEE PERFORMANCE EVALUATION WAS PRESENTED. AREAS OF CONCERN WERE DISCUSSED WITH CORPORATE MANAGEMENT. THE PERFORMANCE AT DRESDEN, QUAD-CITIES AND ZION NUCLEAR GENERATING STATIONS WAS CONSIDERED TO BE ADEQUATE.

REVIEWED BY OMPA LOEB: John J. Rizzo **DATE:** 1-16-81

REVIEWED BY NRR DL: RBBevan **DATE:** 1/19/81

QUAD CITIES 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-265
2. Utility Contact: ROBERT TUBBS (309) 654-2241 X174
3. Licensed Thermal Power (MWt): 2511
4. Nameplate Rating (Gross MWe): 920 x 0.9 = 828
5. Design Electrical Rating (Net MWe): 789
6. Maximum Dependable Capacity (Gross MWe): 813
7. Maximum Dependable Capacity (Net MWe): 769
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	8,040.0	74,110.0
12. Hours Reactor Was Critical	623.0	5,056.8	58,185.7
13. Reactor Reserve Shutdown Hours	.0	.0	2,985.8
14. Hours Generator On-Line	590.6	4,869.7	55,641.5
15. Unit Reserve Shutdown Hours	.0	.0	702.9
16. Gross Thermal Energy (MWH)	1,306,085	10,852,830	113,868,939
17. Gross Electrical Energy (MWH)	426,374	3,425,110	36,288,160
18. Net Electrical Energy (MWH)	395,711	3,212,980	33,955,505
19. Unit Service Factor	82.0	60.6	75.1
20. Unit Availability Factor	82.0	60.6	76.1
21. Unit Capacity Factor (MDC Net)	71.5	52.0	59.6
22. Unit Capacity Factor (DER Net)	69.7	50.6	58.1
23. Unit Forced Outage Rate	18.0	7.1	9.2

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type¹	Duration (Hours)	Reason²	Method of Shutting Down Reactor³	Licensee Event Report No.	System Code⁴	Component Code⁵	Cause & Corrective Action to Prevent Recurrence
19	11/02/80	F	15.9	G	3		CB	INSTRU	REACTOR SCRAMMED ON APRM HIGH FLUX, DUE TO RECIRC PUMP MG SET SPEED TRANSIENT
20	11/03/80	F	91.1	A	1		CB	VALVEX	SHUTDOWN TO REPAIR LEAKING RECIRC SUCTION VALVE, AND PERFORM OTHER MISCELLANEOUS WORK ITEMS
21	11/16/80	F	.0	A	5	80-32	SF	VALVEX	INITIATED SHUTDOWN DUE TO SIMULTANEOUS MALFUNCTIONS IN THE RCIC AND HPCI SYSTEMS LOAD REDUCED TO 360 MWE
22	11/22/80	F	22.4	A	1		CB	VALVEX	REACTOR SHUTDOWN TO REPAIR LEAKAGE RECIRC SUCTION VALVE

SUMMARY

QUAD CITIES 2 OPERATED DURING NOVEMBER WITH 3 OUTAGES FOR EQUIPMENT FAILURE AND 1 OUTAGE FOR OPERATIONAL ERROR.

LEGEND

¹Type:
F--Forced
S--Scheduled

²Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

³Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

⁴Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

⁵Exhibit H:
Same source.

RANCHO SECO

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	California/Sacramento 25 miles SE of Sacramento
2.	Electric Reliability Council	Western Systems Coordinating Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2772
5.	Design Electrical Rating (MWE Net)	918
6.	Date of Initial Criticality	September 16, 1974
7.	Date of Commercial Operation	April 17, 1975 (92.6% of full power)
8.	Condenser Cooling Method	Closed Cycle
9.	Condenser Cooling Water Source	Natural Draft Cooling Towers - Folsom Canal

Utility & Contractor Information

10.	Licensee	Sacramento Municipal Utility District
11.	Corporate Address	6201 South Street P. O. Box 15830 Sacramento, California
12.	Corporate Contact	J. J. Mattimoe Assistant General Manager, Chief Engineer
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	B&W
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	V
17.	IE Branch Chief	J. L. Crews
18.	IE Section Chief	B. H. Faulkenberry
19.	IE Resident Inspector	H. L. Canter (209) 748-2791
20.	Licensing Project Manager	M. L. Padovan
21.	Docket Number	50-312
22.	License Number/Date of Issue	DPR-54, 8/16/74
23.	Public Document Room	Business and Municipal Department Sacramento City-County Library 828 I Street Sacramento, California 95814

R-2

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

+ AWAITING DECISION OF ASLB.

PLANT STATUS:

OPERATION AT OR NEAR FULL POWER FOR MOST OF PERIOD.

LAST IE SITE INSPECTION DATE: 11/12-19/80+

INSPECTION REPORT NO. 50-312/80-31+

SUMMARY

INSPECTION SUMMARY

+ INSPECTION ON NOVEMBER 12 - 19, 1980 (REPORT NO. 50-312/80-31) REPORT BEING PREPARED; TO BE REPORTED NEXT MONTH.

+ INSPECTION ON SEPTEMBER 22 - OCTOBER 3, 1980 (REPORT NO. 50-312/80-32) REPORT BEING PREPARED; TO BE REPORTED NEXT MONTH.

+ INSPECTION ON OCTOBER 1 - 31, 1980 (REPORT NO. 50-312/80-33) AREAS INSPECTED: ROUTINE OPERATIONAL SAFETY VERIFICATION; MONTHLY MAINTENANCE OBSERVATION; MONTHLY SURVEILLANCE OBSERVATION; REVIEW OF PLANT OPERATIONS; FOLLOWUP ON REGIONAL REQUESTS; FOLLOWUP ON HEADQUARTERS' REQUESTS; FOLLOWUP ON SIGNIFICANT EVENTS; REVIEW OF PLANT TRIPS; AND INDEPENDENT INSPECTION EFFORT. THE INSPECTION INVOLVED 162 INSPECTOR HOURS ONSITE BY TWO NRC RESIDENT INSPECTORS.

RESULTS: NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

+ INSPECTION ON OCTOBER 28 - NOVEMBER 7, 1980 (REPORT NO. 50-312/80-34) REPORT BEING PREPARED; TO BE REPORTED NEXT MONTH.

NRC POWER RESTRICTIONS: None

REPORTS RECEIVED FROM LICENSEE

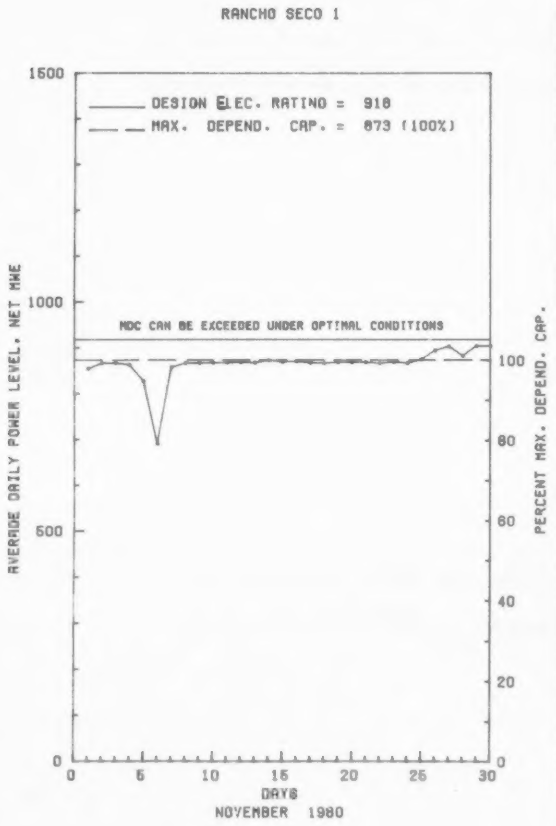
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE			

REVIEWED BY OMPA LOEB: PE Bobe **DATE:** 1/16/81

REVIEWED BY NRR DL: MLPadovan **DATE:** 1/16/81

RANCHO SECO

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-312
2. Utility Contact: R. HOLLINGSWORTH (916) 452-3211
3. Licensed Thermal Power (MWt): 2772
4. Nameplate Rating (Gross MWe): 1070 x 0.9 = 963
5. Design Electrical Rating (Net MWe): 918
6. Maximum Dependable Capacity (Gross MWe): 917
7. Maximum Dependable Capacity (Net MWe): 873
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	8,039.0	53,297.0
12. Hours Reactor Was Critical	720.0	4,679.7	30,752.2
13. Reactor Reserve Shutdown Hours	.0	674.5	4,462.6
14. Hours Generator On-Line	720.0	4,560.3	29,501.6
15. Unit Reserve Shutdown Hours	.0	.0	1,210.2
16. Gross Thermal Energy (MWH)	1,916,865	11,892,223	74,915,571
17. Gross Electrical Energy (MWH)	659,698	3,965,599	25,250,125
18. Net Electrical Energy (MWH)	613,412	3,748,318	23,859,208
19. Unit Service Factor	100.0	56.7	59.8
20. Unit Availability Factor	100.0	56.7	62.3
21. Unit Capacity Factor (MDC Net)	97.6	53.4	55.4
22. Unit Capacity Factor (DER Net)	92.8	50.8	52.7
23. Unit Forced Outage Rate	.0	12.0	29.6

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUELING OUTAGE IN MAY, 1981 - 45 DAYS
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
9	11/05/80	F	.0	A	5		CH	PUMPXX	REDUCED POWER TO REPAIR MAIN FEED PUMP LEAK.

SUMMARY

THE UNIT WAS IN CONTINUOUS OPERATION DURING THE MONTH OF NOVEMBER. AT 2000 ON 11/5, POWER WAS REDUCED TO 70% TO REPAIR A "B" MAIN FEED PUMP LEAK. FULL POWER WAS RESTORED AT 1441 ON 11/6. AT 1630 ON 11/6 A 2 HOUR HOLD AT 88% POWER OCCURRED.

LEGEND

[1] Type:
F–Forced
S–Scheduled

[2] Reason:
A–Equipment Failure
B–Maintenance or Test
C–Refueling
D–Regulatory Restriction
E–Operator Training & License Examination
F–Administrative
G–Operational Error
H–Other

[3] Method:
1–Manual
2–Manual Scram
3–Automatic Scram
4–Continuations
5–Load Reduction
9–Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

best copy available

ROBINSON 2

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	South Carolina/Darlington 5miles NW of Hartsville
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2300
5.	Design Electrical Rating (MWE Net)	700
6.	Date of Initial Criticality	September 20, 1970
7.	Date of Commercial Operation	March 7, 1971
8.	Condenser Cooling Method	Recirculation from impounded lake
9.	Condenser Cooling Water Source	Robinson Impoundment

Utility & Contractor Information

10.	Licensee	Carolina Power and Light Company
11.	Corporate Address	411 Fayetteville Street Raleigh, North Carolina
12.	Corporate Contact	J. A. Jones Executive Vice President
13.	Architect/Engineer	Ebasco
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Ebasco

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R.C. Lewis
18.	IE Section Chief	P. J. Kellogg
19.	IE Resident Inspector	A. K. Hardin
20.	Licensing Project Manager	D. Neighbors
21.	Docket Number	50-261
22.	License Number/Date of Issue	DPR-23, 7/31/70 (5 Mwt), Full power authorization issued 9/23/70
23.	Public Document Room	Hartsville Memorial Library Home and Fifth Avenue Hartsville, South Carolina 29550

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

+ SHUTDOWN FOR PRIMARY COOLANT SYSTEM LEAK REPAIR AT MONTH'S END.

+ LAST IE SITE INSPECTION DATE: NOVEMBER 4-7, 1980

INSPECTION REPORT NO: 50-261/80-36

SUMMARY

INSPECTION SUMMARY

+ INSPECTION OCTOBER 14-17 (80-29): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 31 INSPECTOR-HOURS ON SITE IN THE AREAS OF INSPECTION AND ENFORCEMENT BULLETINS AND REACTOR OPERATIONS. OF THE TWO AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION OCTOBER 20-24 (80-32): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 35 INSPECTOR-HOURS ON SITE. THE AREAS INSPECTED INCLUDED PREPARATION FOR UNIT 2 RESTART, INITIAL CRITICALITY, AND ZERO-POWER PHYSICS TESTS. NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION NOVEMBER 26 (80-33): MATERIAL CONTROL AND ACCOUNTING. THE INSPECTION INVOLVED 16 INSPECTOR-HOURS ON SITE BY ONE NRC INSPECTOR AND WAS BEGUN DURING THE REGULAR HOURS. THE LICENSEE WAS FOUND TO BE IN COMPLIANCE WITH NRC REQUIREMENTS IN THE ONE AREA EXAMINED DURING THE INSPECTION.

INSPECTION OCTOBER 27-31 (80-35): THIS ROUTINE, ANNOUNCED INSPECTION INVOLVED 35 INSPECTOR-HOURS ON SITE IN THE AREAS OF WITNESSING GENERAL PLANT OPERATIONS AND FOLLOWUP INSPECTIONS OF OUTSTANDING ITEMS. OF THE TWO AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

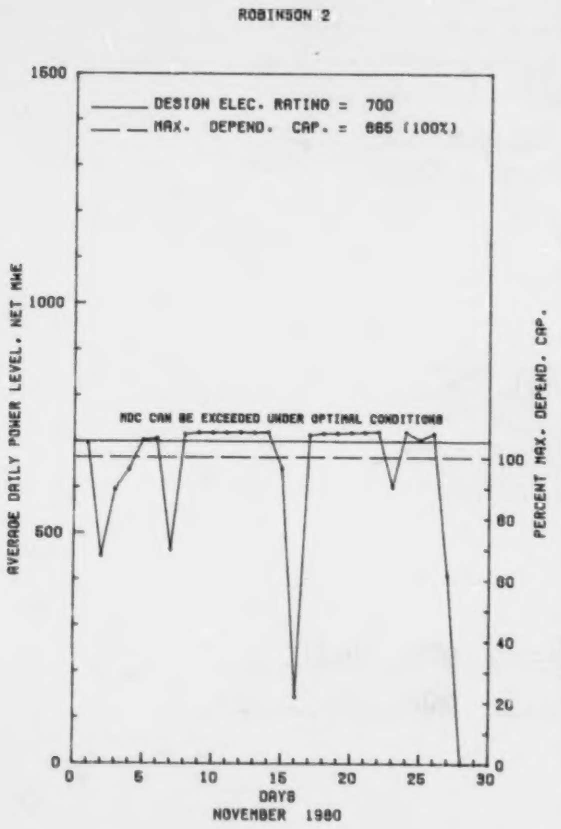
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-024/ 01T-0	10/14/80	10/28/80	PERSONNEL HATCH AIR LOCK DOORS OPEN AT SAME TIME.
80-025/ 01T-0	10/21/80	11/04/80	FAILED WBFD RELAYS
80-026/ 01T-0	10/30/80	11/13/80	NAOH VALVE TO SPRAY PUMPS SHUT WITH REACTOR POWER AT 68 PERCENT

REVIEWED BY OMPA LOEB: Sheryl Massaro **DATE:** 1/16/81

REVIEWED BY NRR DL: Don Neighbors **DATE:** 1/16/81

ROBINSON 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-261
2. Utility Contact: M. L. Watford (803) 383-4524
3. Licensed Thermal Power (MWt): 2300
4. Nameplate Rating (Gross MWe): 854 x 0.9 = 769
5. Design Electrical Rating (Net MWe): 700
6. Maximum Dependable Capacity (Gross MWe): 700
7. Maximum Dependable Capacity (Net MWe): 665
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	8,040.0	85,472.0
12. Hours Reactor Was Critical	623.3	5,122.2	65,329.1
13. Reactor Reserve Shutdown Hours	15.3	238.6	957.7
14. Hours Generator On-Line	605.0	5,039.4	63,730.3
15. Unit Reserve Shutdown Hours	.0	.0	23.2
16. Gross Thermal Energy (MWH)	1,359,962	9,745,780	130,296,109
17. Gross Electrical Energy (MWH)	442,284	3,101,875	42,038,388
18. Net Electrical Energy (MWH)	421,511	2,925,466	39,830,568
19. Unit Service Factor	84.0	62.7	74.6
20. Unit Availability Factor	84.0	62.7	74.6
21. Unit Capacity Factor (MDC Net)	88.0	54.7	70.1
22. Unit Capacity Factor (DER Net)	83.6	52.0	66.5
23. Unit Forced Outage Rate	16.0	18.5	13.6

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
STEAM GENERATOR INSPECTION, APRIL - MAY 1981, 3 WEEKS
25. If Shut Down At End of Report Period, Estimated Date of Startup: 12/06/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
11-01	11/02/80	F	8.0	A	3		HH	VALVEX	FEED FLOW/STEAM FLOW MISMATCH DUE TO FEEDWATER REGULATOR VALVE
11-02	11/07/80	F	6.7	G	3		HC	INSTRU	LOSS OF CONDENSATE AND FEEDWATER PPS UPON BUMPING OF HOTWELL LEVEL SWITCH
11-03	11/15/80	F	18.6	B	1		HA	TURBIN	SHUT DOWN TO BALANCE TURBINE AND "B" REACTOR COOLANT PP
11-04	11/23/80	S	.0	A	5		HH	PUMPXX	LOAD REDUCTION TO REPLACE "B" MAIN FW PP OIL COOLER DUE TO SERVICE WATER LEAK
11-05	11/27/80	F	81.7	A	1		PC	VALVEX	SHUT DOWN TO REPAIR CVC-311 PACKING LEAK. WHILE SHUT DOWN PROBLEMS WERE EXPERIENCED WITH THE NUCLEAR INSTRUMENTATION SYSTEM A CONTROL ROD, AND THE CHARGING SYSTEM. THIS CAUSED AN EXTENSION TO THE INITIAL OUTAGE TIME

SUMMARY

THE UNIT EXPERIENCED 4 OUTAGES AND 1 LOAD REDUCTION DURING THE MONTH OF NOVEMBER. ROBINSON-2 WAS SHUTDOWN AT THE END OF THE MONTH.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

SALEM 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	New Jersey/Salem 20 miles S of Wilmington, Del.
2.	Electric Reliability Council	Mid-Atlantic Area Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	3338
5.	Design Electrical Rating (MWE Net)	1090
6.	Date of Initial Criticality	December 11, 1976
7.	Date of Commercial Operation	June 30, 1977
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Delaware River

Utility & Contractor Information

10.	Licensee	Public Service Electric & Gas Co.
11.	Corporate Address	80 Park Place - 15A Newark, New Jersey 07101
12.	Corporate Contact	F. P. Librizzi General Manager - Electric Prod.
13.	Architect/Engineer	PSE&G
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	UE&C

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	IE Section Chief	R. R. Keimig
19.	IE Resident Inspector	L. J. Norrholm (609-935-3850)
20.	Licensing Project Manager	W. Ross
21.	Docket Number	50-272
22.	License Number/Date of Issue	DPR-70, 8/13/76
23.	Public Document Room	Salem Free Public Library 112 West Broadway Salem, New Jersey 08079

INSPECTION STATUS

ENFORCEMENT STATUS

AMENDMENT 21 TO LICENSE NO. DPR-70 PARAGRAPH C5 STATES THAT "THE APPROVED FIRE PROTECTION PLAN CONSISTS OF ...INFORMATION SUBMITTED WITH LETTERS OF ... MARCH 2, 1979." THE LICENSEE'S LETTER DATED MARCH 2, 1979, TO THE NUCLEAR REGULATORY COMMISSION IN RESPONSE TO REQUEST FOR ADDITIONAL INFORMATION RELATED TO FIRE PROTECTION REVIEW NO. 1 AND 2 UNITS SALEM NUCLEAR GENERATING STATION, STATES THAT "WE COMPLY WITH THE SUPPLEMENTAL GUIDANCE CONTAINED IN 'NUCLEAR PLANT FIRE PROTECTION FUNCTIONAL RESPONSIBILITIES ADMINISTRATION CONTROLS AND QUALITY ASSURANCE,' DATED JUNE 14, 1977." NRC DOCUMENT TITLED, NUCLEAR PLANT FIRE PROTECTION FUNCTIONAL RESPONSIBILITIES, ATTACHMENT NO. 2, FIRE BRIGADE TRAINING STATES: "THE TRAINING PROGRAM SHOULD CONSIST OF...CLASSROOM INSTRUCTIONS, PRACTICE IN FIRE FIGHTING...USE OF EMERGENCY BREATHING EQUIPMENT.... INSTRUCTIONS SHOULD BE PROVIDED TO ALL FIRE BRIGADE MEMBERS...." CONTRARY TO THE ABOVE, THE LICENSEE'S RECORDS INDICATED THAT APPROXIMATELY ONE-HALF OF THE LISTED FIRE BRIGADE MEMBERS HAVE NOT RECEIVED TRAINING IN FIRE FIGHTING AND IN THE USE OF EMERGENCY BREATHING EQUIPMENT. THE LICENSEE'S RECORDS ALSO INDICATED THAT ON SEPTEMBER 18 AND 19, 1980, ONE OUT OF THE FIVE PERSONS ASSIGNED TO THE FIRE BRIGADE HAD NO PREVIOUS RELATED TRAINING. ON OCTOBER 14, 1980 ONE PERSON ASSIGNED TO THE FIRE BRIGADE WAS NOT QUALIFIED ON EMERGENCY BREATHING EQUIPMENT. THIS ITEM REFERS TO DOCKET NOS. 50-272 AND 50-311.
(8027 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEM AND COMPONENT PROBLEMS:

- THE LICENSEE HAS IDENTIFIED DESIGN PROBLEMS WITH SEVERAL MASONRY WALLS THAT WERE NOT PREVIOUSLY CONSIDERED IN THEIR RESPONSE TO IE BULLETIN 80-11.

FACILITY ITEMS (PLANS AND PROCEDURES):

- DESIGN CHANGES AND MAINTENANCE SCHEDULED FOR THIS REFUELING OUTAGE ARE IN PROGRESS. STARTUP AFTER REFUELING MAY BE DELAYED UNTIL ISSUES INVOLVING MASONRY WALLS ARE RESOLVED.
- MANAGERIAL ITEMS:
- THE LICENSEE HAS ANNOUNCED A MANAGEMENT REORGANIZATION AT THE CORPORATE LEVEL. THE REORGANIZATION INCLUDES THE FOLLOWING CHANGES:
- A. RESPONSIBILITIES FOR NUCLEAR AND FOSSIL PLANTS HAVE BEEN SEGREGATED AND ASSIGNED TO SEPARATE GENERAL MANAGERS.
- B. GENERAL MANAGER FOR CORPORATE QUALITY ASSURANCE REPORTS DIRECTLY TO THE SENIOR VICE-PRESIDENT - ENERGY SUPPLY AND ENGINEERING.

PLANT STATUS:

- REFUELING IS COMPLETE. PLANT IS IN MODE 5 WITH RESTART PLANNED FOR MID-DECEMBER.
- LAST IE SITE INSPECTION DATE: 11/17-25/80
- INSPECTION REPORT NO: 50-272/80-31

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-048/ 03L	08/30/80	09/29/80	QUADRANT POWER TILT RATIO EXCEEDS 1.02 WITH 67 PERCENT THERMAL POWER
80-049/ 03L	08/30/80	09/29/80	ECCS SUBSYSTEM INOPERABLE - NO. 12 CENTRIFUGAL CHARGING PUMP LUBE OIL COOLER SERVICE WATER LEAK
80-050/ 03L	09/03/80	10/03/80	AIR PARTICULATE MONITOR 1R11A FAILED - REPLACED DETECTOR
80-053/ 03L	10/20/80	10/28/80	INADVERTENT DILUTION OF REACTOR COOLANT SYSTEM DURING STEAM GENERATOR DECONTAMINATION
80-054/ 01T	10/16/80	10/30/80	FIRE SUPPRESSION WATER PUMPS-BOTH PUMPS OUT OF SERVICE
80-055/ 01T	10/29/80	11/14/80	STEAM GENERATOR NO. 13 FEED NOZZLE TO PIPE WELD, RADIOGRAPH REVEALED SEVERAL INDICATIONS REQUIRING FURTHER EXAMINATIONS
80-056/ 01P	11/07/80	11/10/80	INVESTIGATION OF MASONRY WALLS PER IE BULLETIN 80-11 REVEALED THREE WALLS WERE NOT CONSTRUCTED AS DESIGNED

SUMMARY

INSPECTION SUMMARY

50-272/80-27 - OCT 14-17: ROUTINE, UNANNOUNCED ONSITE INSPECTION BY ONE REGION-BASED INSPECTOR (15 HOURS) OF THE FACILITY'S FIRE PROTECTION/PREVENTION PROGRAM INCLUDING: ADMINISTRATIVE CONTROLS; FIRE BRIGADE TRAINING; REVIEW AND OBSERVATION OF PLANT MODIFICATIONS; AND OBSERVATION OF CRITICAL PLANT FIRE AREAS. ONE ITEM OF NONCOMPLIANCE: FAILURE TO ADEQUATELY TRAIN FIRE BRIGADE.

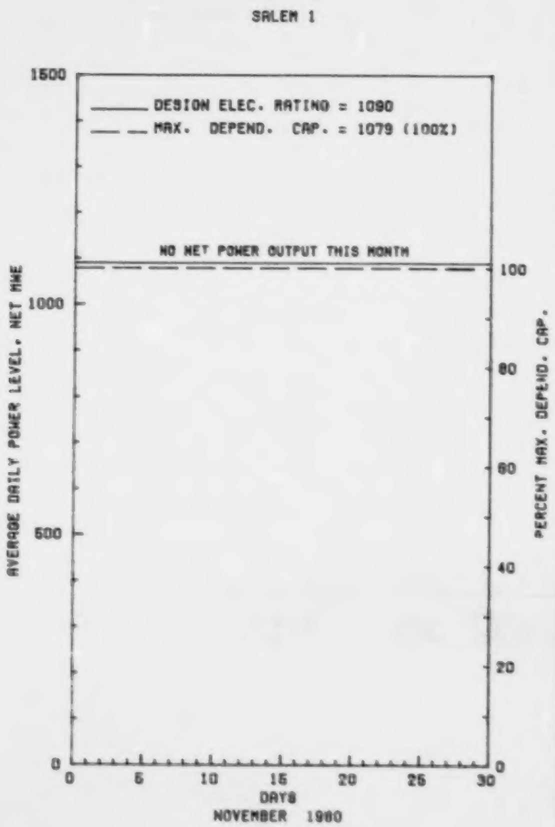
50-272/80-29 - NOV 5-6: ROUTINE, ANNOUNCED ONSITE INSPECTION BY A REGION-BASED INSPECTOR (12 HOURS) OF COMPONENTS ASSOCIATED WITH THE COMPONENT COOLING WATER SYSTEM, REACTOR COOLANT PUMP, DRAIN TANK, LETDOWN HEAT EXCHANGER FLOW AND DISCHARGE CONTROL PANELS IN RESPONSE TO NRC BULLETIN NO. 79-01B. NO NONCOMPLIANCES WERE IDENTIFIED.

REVIEWED BY OMPA LOEB: Steve J Anussaro **DATE:** 1/16/81

REVIEWED BY NRR DL: W J Ross **DATE:** 1/19/81

SALEM 1

AVERAGE DAILY POWER LEVEL(MWe)



OPERATING STATUS

1. Docket Number: 50-272
2. Utility Contact: L. K. Miller (609) 365-7000
3. Licensed Thermal Power (MWt): 3338
4. Nameplate Rating (Gross MWe): 1300 x 0.9 = 1170
5. Design Electrical Rating (Net MWe): 1090
6. Maximum Dependable Capacity (Gross MWe): 1124
7. Maximum Dependable Capacity (Net MWe): 1079
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	8,040.0	30,001.0
12. Hours Reactor Was Critical	.0	6,097.8	16,186.1
13. Reactor Reserve Shutdown Hours	.0	.0	22.7
14. Hours Generator On-Line	.0	5,946.3	15,477.4
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	18,285,037	45,741,333
17. Gross Electrical Energy (MWH)	0	5,935,290	15,102,590
18. Net Electrical Energy (MWH)	-2,812	5,658,279	14,287,069
19. Unit Service Factor	.0	74.0	51.6
20. Unit Availability Factor	.0	74.0	51.6
21. Unit Capacity Factor (MDC Net)	.0	65.2	44.1
22. Unit Capacity Factor (DER Net)	.0	64.6	43.7
23. Unit Forced Outage Rate	.0	5.8	34.4

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 12/25/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-210	09/19/80	S	720.0	C	4		RC	FUELXX	ANNUAL REFUELING OUTAGE CONTINUES.

SUMMARY

THE UNIT CONTINUED IN REFUELING OUTAGE (1730 HOURS TO-DATE). THE OUTAGE IS SCHEDULED TO END ON 12/25/80.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

SAN ONOFRE 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	California/San Diego 5 miles S of San Clemente
2.	Electric Reliability Council	Western Systems Coordinating Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	1347
5.	Design Electrical Rating (MWE Net)	436
6.	Date of Initial Criticality	June 14, 1967
7.	Date of Commercial Operation	January 1, 1968
8.	Condenser Cooling Method	Circulating sea water
9.	Condenser Cooling Water Source	Pacific Ocean

Utility & Contractor Information

10.	Licensee	Southern California Edison Co.
11.	Corporate Address	2244 Walnut Grove Avenue Rosemead, California 91770
12.	Corporate Contact	R. E. Woodbury V.P. & General Counsel
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	V
17.	IE Branch Chief	J. L. Crews
18.	IE Section Chief	B. H. Faulkenberry
19.	IE Resident Inspector	L. F. MILLER (714-492-2641)
20.	Licensing Project Manager	S. J. Nowicki
21.	Docket Number	50-206
22.	License Number/Date of Issue	DPR-13, 3/27/67
23.	Public Document Room	Mission Viejo Branch Library 24851 Chrisanta Drive Mission Viejo, California 92676

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

SYSTEMS AND COMPONENT PROBLEMS:

LARGE NUMBER OF TUBES DETERMINED TO NEED PLUGGING (RESULTS OF EDDY CURRENT TESTING). LICENSEE PLANS TO INSTALL SLEEVES IN TUBES IN LIEU OF PLUGGING.

FACILITY ITEMS (PLANS AND PROCEDURES):

STEAM GENERATOR REPAIRS.

MANAGERIAL ITEMS

NONE.

PLANT STATUS:

PLANT SHUTDOWN FOR STEAM GENERATOR REPAIRS

LAST IE SITE INSPECTION DATE: 11/03-07/80

INSPECTION REPORT NO: 50-206/80-29

SUMMARY

INSPECTION SUMMARY

- INSPECTION ON SEPTEMBER 22 - 26, 1980 (REPORT NO. 50-206/80-26) REPORT SENT TO HEADQUARTERS FOR REVIEW.
- INSPECTION ON NOVEMBER 3 - 7, 1980 (REPORT NO. 50-206/80-29) REPORT BEING PREPARED; TO BE REPORTED NEXT MONTH.
- INSPECTION ON SEPTEMBER 29 - OCTOBER 31, 1980 (REPORT NO. 50-206/80-31) AREAS INSPECTED: ROUTINE RESIDENT INSPECTION OF PLANT OPERATIONS DURING LONG TERM OUTAGE: MONTHLY MAINTENANCE AND SURVEILLANCE OBSERVATIONS; FOLLOWUP ON ENFORCEMENT ITEMS; FOLLOWUP ON LICENSEE RESPONSES TO IE BULLETINS; TMI SHORT TERM LESSONS LEARNED VERIFICATION (TEMPORARY INSTRUCTIONS 2515/42, 43 AND 44); AND FOLLOWUP ON A SIGNIFICANT EVENT. (LOSS OF FOREIGN MATERIAL CONTROL INSIDE REACTOR COOLANT SYSTEM). THIS INSPECTION INVOLVED 70 INSPECTOR HOURS ONSITE BY THE NRC RESIDENT INSPECTOR.

  RESULTS: NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.
- INSPECTION ON NOVEMBER 3 - 28, 1980 (REPORT NO. 50-206/80-32) REPORT BEING PREPARED; TO BE REPORTED NEXT MONTH.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

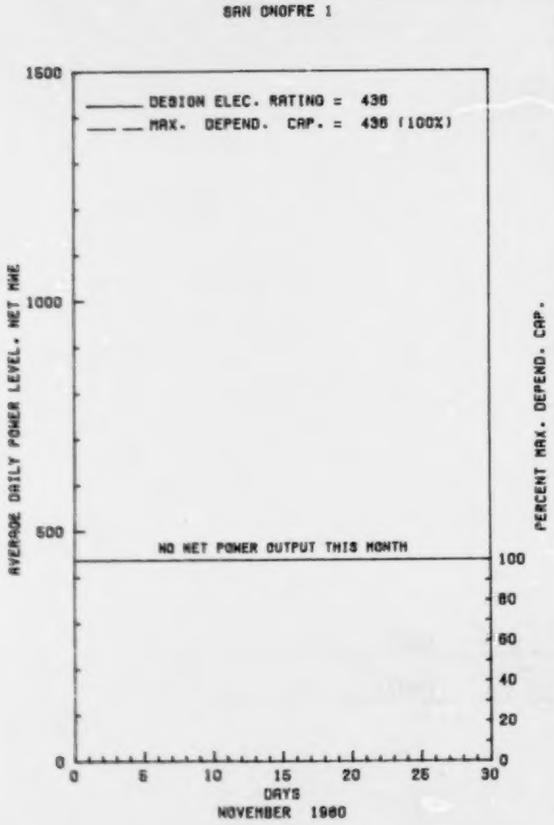
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE			

REVIEWED BY OMPA LOEB: DATE: 1/16/81

REVIEWED BY NRR DL: S. J. Nowicki DATE: 1/19/81

SAN ONOFRE 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-206
2. Utility Contact: J. M. JOY (714) 492-7700
3. Licensed Thermal Power (MWt): 1347
4. Nameplate Rating (Gross MWe): 500 x 0.9 = 450
5. Design Electrical Rating (Net MWe): 436
6. Maximum Dependable Capacity (Gross MWe): 456
7. Maximum Dependable Capacity (Net MWe): 436
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

CAPACITY FACTORS CALCULATED WITH A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	8,040.0	118,016.0
12. Hours Reactor Was Critical	.0	1,979.1	84,611.3
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	.0	1,961.6	81,116.0
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	2,552,787	104,084,261
17. Gross Electrical Energy (MWH)	0	855,000	35,526,435
18. Net Electrical Energy (MWH)	0	816,676	33,664,042
19. Unit Service Factor	.0	24.4	68.7
20. Unit Availability Factor	.0	24.4	68.7
*21. Unit Capacity Factor (MDC Net)	.0	23.3	69.8
22. Unit Capacity Factor (DER Net)	.0	23.3	69.8
23. Unit Forced Outage Rate	100.0	63.9	12.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 04/16/81

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
55	07/12/80	F	720.0	A	4	80-014	CC	HTEXCH	STEAM GENERATOR TUBE REPAIR CONTINUES.

SUMMARY

STEAM GENERATOR TUBE REPAIR CONTINUED DURING THE MONTH OF NOVEMBER.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

SEQUOYAH 1

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Tennessee/Hamilton 9.5 Miles NE of Chattanooga
2.	Electric Reliability Council	SERC
3.	Type of Reactor	PWR
4.	Licensed Power Level (MWT)	3411
5.	Design Electrical Rating (MWE Net)	1140
6.	Date of Initial Criticality	July 5, 1980
7.	Date of Commercial Operation	N/A
8.	Condenser Cooling Method	Natural Draft Cooling Towers
9.	Condenser Cooling Water Source	Tennessee River

Utility and Contractor Information

10.	Licensee	Tennessee Valley Authority
11.	Corporate Address	500A Chestnut Street Tower II Chattanooga, TN 37401
12.	Corporate Contact	H. G. Parris, Manager of Power
13.	Architect/Engineer	TVA
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	TVA

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. Lewis
18.	IE Section Chief	H. C. Dance
19.	IE Resident Inspector	S. D. BUTLER
20.	Licensing Project Manager	C. R. Stahle
21.	Docket Number	50-327
22.	License Number/Date of Issue	DPR-77, 9/17/80
23.	Public Document Room	Chattanooga - Hamilton Bicentennial Library 1001 Broad Street Chattanooga, Tennessee 37402

INSPECTION STATUS

ENFORCEMENT STATUS

CONTRARY TO THE TECHNICAL SPECIFICATION 6.9.1.11, THE LICENSEE SUBMITTED THREE 24 HOUR, ONE 14 DAY, AND TWENTY TWO 30 DAY REPORTS LATE. ALSO CONTRARY TO TECHNICAL SPECIFICATION 4.8.1.1.4, TEN DIESEL GENERATOR REPORTS DID NOT INCLUDE ALL THE INFORMATION REQUIRED BY REGULATORY GUIDE 1.108.
(8039 INFRACTION)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE

MANAGERIAL ITEMS:

NONE

PLANT STATUS:

STARTUP TEST PROGRAM.

◆ LAST IE SITE INSPECTION DATE: NOVEMBER 4-30, 1980

◆ INSPECTION REPORT NO: 50-327/80-47

SUMMARY

INSPECTION SUMMARY

◆ INSPECTION SEPTEMBER 1-30 (80-38): THIS ROUTINE ANNOUNCED INSPECTION INVOLVED 84 INSPECTOR-HOURS ON SITE IN THE AREAS OF LICENSEE ACTION ON PREVIOUS INSPECTION FINDINGS, OPERATIONAL SAFETY VERIFICATION OF LICENSE CONDITIONS, LICENSEE EVENT REPORT REVIEW, INDEPENDENT INSPECTION EFFORT AND FOLLOWUP ON PLANT INCIDENTS. OF THE SIX AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN FIVE AREAS; ONE ITEM OF NONCOMPLIANCE WAS FOUND IN ONE AREA (DEFICIENCY - IMPROPERLY MAINTAINING AND MAKING CHANGES TO A PROCEDURE REQUIRED BY TECHNICAL SPECIFICATION)

INSPECTION OCTOBER 6-7 (80-40): INCLUDED REVIEW OF LICENSEE IDENTIFIED ITEM OF NONCOMPLIANCE AND LICENSEE PROCEDURE FOR UNESCORTED ACCESS CLEARANCES. THE INSPECTOR ALSO REVIEWED THE LICENSEES INVESTIGATION OF A RECENT INTRUSION OF THE PROTECTED AREA AT BROWNS FERRY. THE INSPECTION WAS BEGUN DURING A REGULAR PERIOD. THE LICENSEE WAS FOUND TO BE IN COMPLIANCE WITH NRC REQUIREMENTS WITHIN THE AREAS EXAMINED DURING THE INSPECTION.

REPORTS RECEIVED FROM LICENSEE

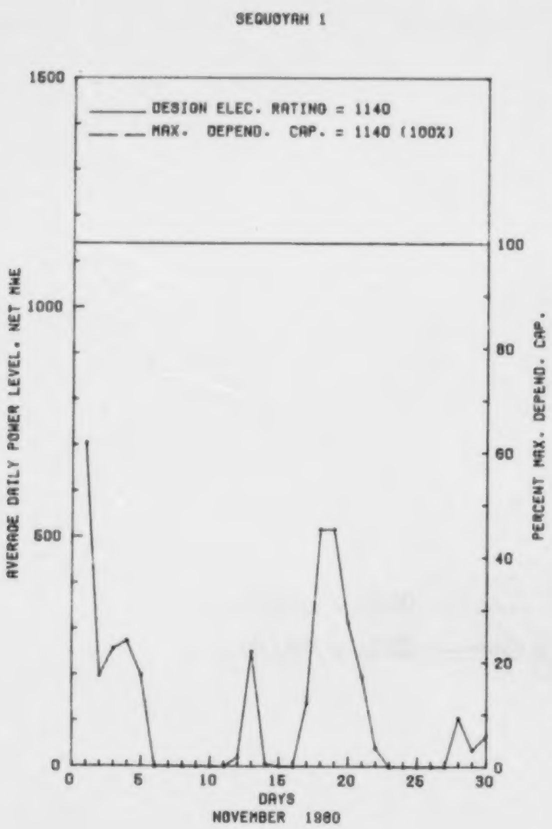
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-180/03L-0	11/14/80	11/18/80	MASONRY BLOCK WALLS FOUND TO NOT BE SEISMICALLY QUALIFIED

REVIEWED BY OMPA LOEB: [illegible] **DATE:** [illegible]

REVIEWED BY NRR DL: [illegible] **DATE:** [illegible]

SEQUOYAH 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-327
2. Utility Contact: DAVID DUPREE (615) 842-0295
3. Licensed Thermal Power (MWt): 3411
4. Nameplate Rating (Gross MWe): 1220.5
5. Design Electrical Rating (Net MWe): 1140
6. Maximum Dependable Capacity (Gross MWe): 1185
7. Maximum Dependable Capacity (Net MWe): 1140
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	6,601.0	6,601.0
12. Hours Reactor Was Critical	330.5	1,016.1	1,016.1
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	300.8	337.2	337.2
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	398,591	467,259	467,259
17. Gross Electrical Energy (MWH)	104,781	109,500	109,500
18. Net Electrical Energy (MWH)	92,324	92,324	92,324
19. Unit Service Factor			
20. Unit Availability Factor			
21. Unit Capacity Factor (MDC Net)			
22. Unit Capacity Factor (DER Net)			
23. Unit Forced Outage Rate			

NOT IN COMMERCIAL OPERATION

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause and Corrective Action to Prevent Recurrence
15	11/05/80	F	136.9	H	3				AN INJECTION PRESSURE SURGE ON MAIN FEEDWATER SYSTEM CAUSED REACTOR TRIP.
16	11/11/80	F	7.0	A	3				VITAL INVERTER VOLTAGE SPIKES CAUSED INDICATED LOW LEVEL IN #2 S/G CAUSING REACTOR TRIP.
17	11/12/80	F	22.9	A	3				MAIN TURBINE GENERATOR BACK-UP RELAY SYSTEM PROBLEM CAUSED REACTOR TRIP.
18	11/14/80	F	84.1	A	3				REACTOR TRIP CAUSED BY CONTAINMENT SPRAY SYSTEM BEING INOPERABLE.
19	11/20/80	F	12.7	A	3				LOW LEVEL IN S/G #3 CAUSED REACTOR TRIP.
20	11/21/80	F	18.7	A	3				SWINGING OF MFPT CONTROLS CAUSED HIGH S/G LEVEL RESULTING IN A REACTOR TRIP.
21	11/22/80	F	1.4	H	3				INSTRUMENTATION PERSONNEL INSTALLING TEST EQUIPMENT IN THE NIS CABINETS CAUSED REACTOR TRIP.
22	11/22/80	F	6.3	H	3				INSTRUMENTATION PERSONNEL INSTALLING TEST EQUIPMENT IN THE NIS CABINETS CAUSED REACTOR TRIP.
23	11/22/80	F	4.2	H	3				A PLANNED NEGATIVE RATE TRIP PER STARTUP TEST SU-9.5 CAUSED TWO RODS TO DROP AND THE TRIP.
24	11/23/80	F	5.0	H	3				TRIP CAUSED BY STARTUP TEST SU-1.2A TO VERIFY THE REACTOR CAN BE SHUTDOWN OUTSIDE THE CONTROL ROOM.
25	11/23/80	F	119.9	H	3				REACTOR TRIP CAUSED BY STARTUP TEST SU-1.1A TO PROVE REACTOR CAN BE SHUTDOWN NORMALLY WITHOUT OFFSITE POWER (BLACKOUT).

SUMMARY

THE UNIT IS IN POWER ASCENSION TESTING AND NUMEROUS OUTAGES WERE EXPERIENCED DURING THE MONTH OF NOVEMBER. COMMERCIAL OPERATION IS SCHEDULED FOR DECEMBER 30, 1980.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & Licensee Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source

ST. LUCIE 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Florida/St. Lucie 12 miles SE of Ft. Pierce
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2560
	Design Electrical Rating (MWE Net)	802
5.	Date of Initial Criticality	April 22, 1976
7.	Date of Commercial Operation	December 21, 1976
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Atlantic Ocean

Utility & Contractor Information

10.	Licensee	Florida Power & Light Co.
11.	Corporate Address	9250 W. Flagler Street P. O. Box 529100 Miami, Florida 33152
12.	Corporate Contact	R. E. Uhrig, V.P.
13.	Architect/Engineer	Ebasco
14.	Nuclear Steam Supply System	Combustion Engineering
15.	Constructor	Ebasco

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. LEWIS
18.	IE Section Chief	R. A. Martin
19.	IE Resident Inspector	S. A. ELROD
20.	Licensing Project Manager	C. Nelson
21.	Docket Number	50-335
22.	License Number/Date of Issue	DPR-67, March 1, 1976
23.	Public Document Room	Indian River Junior College Library 3209 Virginia Avenue Ft. Pierce, Florida 33450

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

NORMAL OPERATION.

+ LAST IE SITE INSPECTION DATE: NOVEMBER 8-28, 1980

+ INSPECTION REPORT NO: 50-335/80-36

SUMMARY

INSPECTION SUMMARY

+ INSPECTION OCTOBER 14-17 (80-32): INCLUDED REVIEW OF SECURITY PROGRAM AUDIT; PHYSICAL BARRIERS (PROTECTED AREA); LIGHTING; ACCESS CONTROL (IDENTIFICATION, AUTHORIZATION AND BADGING, AND ESCORTING); ALARM STATIONS; DETECTION AIDS; COMMUNICATIONS; TESTING AND MAINTENANCE; LOCKS, KEYS, AND COMBINATIONS; AND RECORDS AND REPORTS, AND ASSESSMENT AIDS. THE INSPECTION INVOLVED 26 INSPECTOR-HOURS ON SITE BY ONE NRC INSPECTOR. THE INSPECTION WAS BEGUN DURING AN OFFSHIFT PERIOD; SIX INSPECTION HOURS WERE ACCOMPLISHED DURING OFFSHIFT PERIODS. THE LICENSEE WAS FOUND TO BE IN COMPLIANCE WITH NRC REQUIREMENTS WITHIN THE 11 AREAS EXAMINED DURING THE INSPECTION EXCEPT FOR THE FOLLOWING 2 ITEMS: (1) PHYSICAL BARRIER (PROTECTED AREA) - FAILURE TO MAINTAIN ISOLATION ZONE CLEAR OF VEGETATION THAT COULD CONCEAL OR SHIELD AN INDIVIDUAL; (2) ACCESS CONTROL (IDENTIFICATION, AUTHORIZATION, AND BADGING) - FAILURE TO REMOVE UNRETRIEVED SECURITY BADGE AND SECURITY KEY CARD FROM THE SECURITY COMPUTER SYSTEM

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

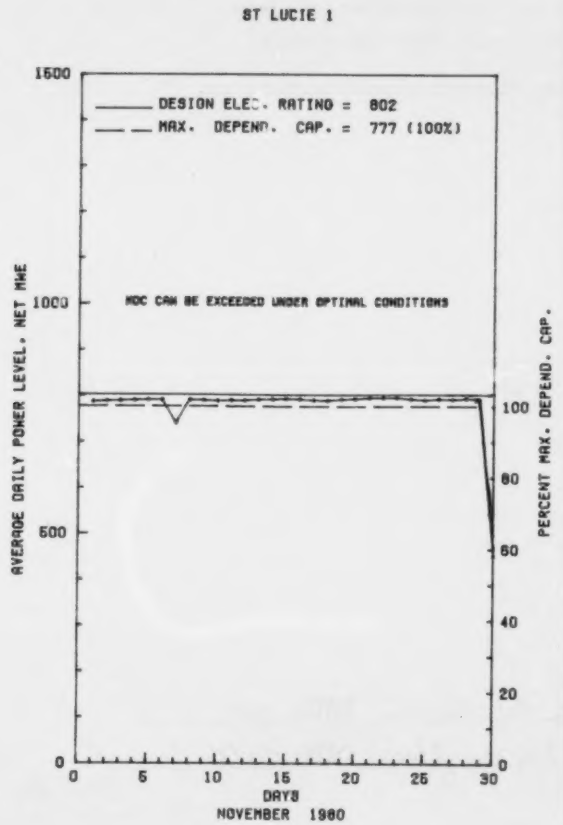
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-057/ 03L-0	10/08/80	11/06/80	FIRE HOSE INOPERABLE AND INTERIM MEASURES NOT IMPLEMENTED
80-060/ 03L-0	09/29/80	10/29/80	CONDENSATE TIE LINE BETWEEN CONDENSATE STORAGE TANK AND AUX. FEED WATER SYSTEM DID NOT MEET DESIGN INTENT

REVIEWED BY OMPA LOEB: MR Beebe **DATE:** 01/16/81

REVIEWED BY NRR DL: C Nelson **DATE:** 01/16/81

ST. LUCIE 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-335
2. Utility Contact: V. T. Chilson (305) 552-3824
3. Licensed Thermal Power (MWt): 2560
4. Nameplate Rating (Gross MWe): 1000 x 0.85 = 850
5. Design Electrical Rating (Net MWe): 802
6. Maximum Dependable Capacity (Gross MWe): 822
7. Maximum Dependable Capacity (Net MWe): 777
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	8,040.0	34,584.0
12. Hours Reactor Was Critical	713.8	6,171.6	27,481.5
13. Reactor Reserve Shutdown Hours	.0	.0	129.4
14. Hours Generator On-line	711.8	6,057.2	26,887.3
15. Unit Reserve Shutdown Hours	.0	7.3	39.3
16. Gross Thermal Energy (MWH)	1,814,059	15,084,052	65,345,618
17. Gross Electrical Energy (MWH)	592,380	4,892,080	21,186,580
18. Net Electrical Energy (MWH)	560,714	4,609,064	19,936,960
19. Unit Service Factor	98.9	75.3	77.7
20. Unit Availability Factor	98.9	75.4	77.9
21. Unit Capacity Factor (MDC Net)	100.2	73.8	74.2
22. Unit Capacity Factor (DER Net)	97.1	71.5	71.9
23. Unit Forced Outage Rate	1.1	7.6	5.9

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
18	11/30/80	F	6.1	A	3		IA	INSTRU	UNIT WAS TRIPPED DUE TO LOSS OF POWER SUPPLY TO THE CONTROL SYSTEM WHEN THE OUTPUT BREAKER OF THE SECOND OF TWO MOTOR-GENERATOR (M-G) SETS TRIPPED OPEN. M-G NO. 1A WAS REMOVED FROM SERVICE WHILE PERFORMING DC GROUND ISOLATION INVESTIGATION.
19	11/30/80	F	2.1	A	3		HH	ZZZZZZ	UNIT WAS TRIPPED BY THE STEAM GENERATOR LEVEL PROTECTION SYSTEM DURING A TRANSIENT CONDITION WHILE INCREASING LOAD AFTER THE UNIT WAS RETURNED TO SERVICE.

SUMMARY

THE UNIT OPERATED AT 100% POWER EXCEPT FOR TWO SHORT DURATION OUTAGES ON NOVEMBER 30. NUREG-0578 IMPLEMENTATION IS CONTINUING.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

SURRY 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Virginia/Surry 17 miles NW of Newport News
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2441
5.	Design Electrical Rating (MWE Net)	822
6.	Date of Initial Criticality	July 1, 1972
7.	Date of Commercial Operation	December 22, 1972
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	James River

Utility & Contractor Information

10.	Licensee	Virginia Electric and Power Company
11.	Corporate Address	P. O. Box 26666 Richmond, Virginia 23261
12.	Corporate Contact	J. H. Ferguson Exec. V.P. - Power
13.	Architect/Engineer	Stone & Webster
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R.C. Lewis
18.	IE Section Chief	P. J. Kellogg
19.	IE Resident Inspector	D. Burke (804) 357-2101
20.	Licensing Project Manager	D. Neighbors
21.	Docket Number	50-280
22.	License Number/Date of Issue	DPR-32, 5/25/72
23.	Public Document Room	Swem Library College of William and Mary Williamsburg, Virginia 23185

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

STEAM GENERATOR TUBE LEAKS.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

UNIT IS CURRENTLY SHUTDOWN FOR STEAM GENERATOR REPLACEMENT.

♦ LAST IE SITE INSPECTION DATE: SEPTEMBER 28 - OCTOBER 2, 1980

INSPECTION REPORT NO: 50-280/80-38

SUMMARY

INSPECTION SUMMARY

♦ INSPECTION SEPTEMBER 28 - OCTOBER 2 (80-38): INCLUDED REVIEW OF THE SECURITY PLAN, PHYSICAL BARRIERS (PROTECTED AREA), PHYSICAL BARRIERS (VITAL AREA), LIGHTING, ACCESS CONTROL (IDENTIFICATION, AUTHORIZATION, AND BADGING), ACCESS CONTROL (SEARCH), ACCESS CONTROL (ESCORTING) AND ACCESS CONTROL (VITAL AREAS). THE INSPECTION INVOLVED 30 INSPECTOR HOURS ON SITE BY ONE NRC INSPECTOR. THE INSPECTION WAS BEGUN DURING AN OFFSHIFT PERIOD; FIVE INSPECTION HOURS WERE ACCOMPLISHED DURING OFFSHIFT PERIODS. THE LICENSEE WAS FOUND TO BE IN COMPLIANCE WITH NRC REQUIREMENTS WITHIN THE EIGHT AREAS EXAMINED DURING THE INSPECTION EXCEPT FOR THE FOLLOWING FOUR ITEMS: (1) ACCESS CONTROL (SEARCH) - NO COMMITMENT IN THE SECURITY PLAN TO A PERCENTAGE HANDS-ON SEARCH OF NON-REGULAR EMPLOYEES; (2) ACCESS CONTROL (SEARCH) - FAILURE TO HAVE COMPENSATORY MEASURES FOR PERSONNEL SEARCH EQUIPMENT; (3) ACCESS CONTROL (VITAL AREAS) - FAILURE TO POSITIVELY CONTROL ALL POINTS OF PERSONNEL ACCESS INTO VITAL AREAS; AND (4) ACCESS CONTROL (VITAL AREAS) - FAILURE TO HAVE ADEQUATE COMPENSATORY PROCEDURES FOR CONTROLLING ACCESS INTO VITAL AREAS.

INSPECTION SEPTEMBER 2 - OCTOBER 3 (80-39): THIS INSPECTION INVOLVED 49 (RESIDENT) INSPECTOR-HOURS ON SITE IN THE AREAS OF PLANT OPERATIONS AND OPERATING RECORDS, PLANT MODIFICATIONS, MAINTENANCE AND TESTING, REFUELING ACTIVITIES, LICENSEE EVENT REPORTS, AND PLANT SECURITY. OF THE SIX AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN FOUR AREAS; TWO ITEMS OF NONCOMPLIANCE WERE FOUND IN TWO AREAS (INFRACTION - FAILURE TO PROPERLY EVALUATE AND DOCUMENT INSTALLATION OF A JUMPER HOSE ON SUMP PIPING; INFRACTION - FAILURE TO PROPERLY SEAL UNIT 1 CONTAINMENT DOOR DURING REFUELING).

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

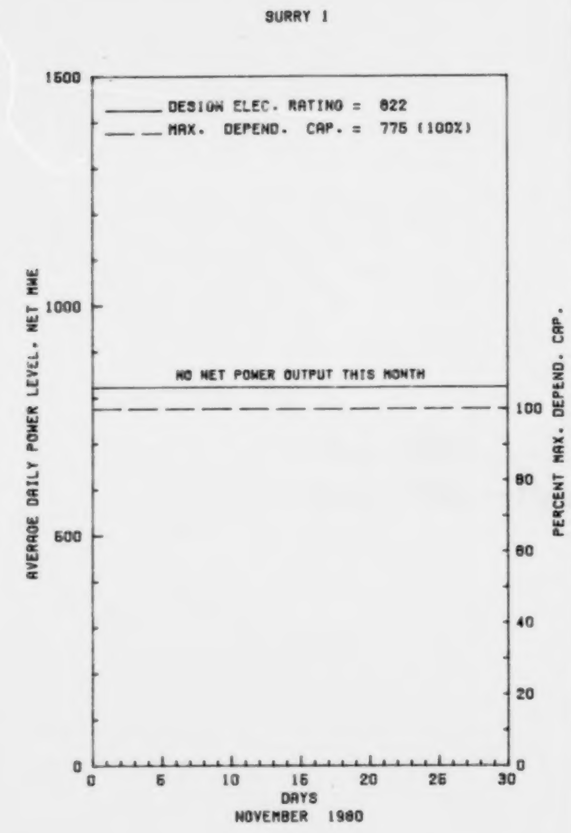
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-059/01T-0	10/16/80	10/30/80	WESTINGHOUSE INFORMED LICENSEE OF POTENTIAL FOR AUX. FW PUMP RUNOUT.
80-067/01T-0	10/28/80	11/10/80	RADIOGRAPHY EXAM ON ALL 3 STEAM GENERATOR STEAM ELBOWS SHOWED INDICATIONS
80-068/03L-0	09/23/80	10/23/80	RADIATION MONITOR PUMPS 1-SW-P-5C AND 5D FOUND INOPERABLE

REVIEWED BY OMPA LOEB: John F. Crooks **DATE:** 1/16/81

REVIEWED BY NRR DL: Don Neighbors **DATE:** 1/16/81

SURRY 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-280
2. Utility Contact: SUE DUNN (804) 357-3184
3. Licensed Thermal Power (MWt): 2441
4. Nameplate Rating (Gross MWe): 942 x 0.9 = 848
5. Design Electrical Rating (Net MWe): 822
6. Maximum Dependable Capacity (Gross MWe): 811
7. Maximum Dependable Capacity (Net MWe): 775
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	8,040.0	69,424.0
12. Hours Reactor Was Critical	0	3,786.9	42,538.5
13. Reactor Reserve Shutdown Hours	0	122.6	3,731.5
14. Hours Generator On-Line	0	3,763.8	41,668.2
15. Unit Reserve Shutdown Hours	0	183.6	3,736.2
16. Gross Thermal Energy (MWH)	0	8,674,529	96,159,401
17. Gross Electrical Energy (MWH)	0	2,621,265	31,391,743
18. Net Electrical Energy (MWH)	0	2,473,025	29,699,926
19. Unit Service Factor	0	46.8	59.8
20. Unit Availability Factor	0	49.1	65.2
*21. Unit Capacity Factor (MDC Net)	0	39.7	55.0
22. Unit Capacity Factor (DER Net)	0	37.4	51.9
23. Unit Forced Outage Rate	0	39.1	26.4

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 09/01/81

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-15	09/14/80	S	720.0	A	4				CONTINUATION OF SHUTDOWN FOR STEAM GENERATOR REPLACEMENT OUTAGE WHICH COMMENCED 09-14-80.

SUMMARY

SURRY 1 REMAINED SHUTDOWN IN NOVEMBER FOR A CONTINUING STEAM GENERATOR REPLACEMENT OUTAGE.

LEGEND

[1] Type:
F–Forced
S–Scheduled

[2] Reason:
A–Equipment Failure
B–Maintenance or Test
C–Refueling
D–Regulatory Restriction
E–Operator Training & License Examination
F–Administrative
G–Operational Error
H–Other

[3] Method:
1–Manual
2–Manual Scram
3–Automatic Scram
4–Continuations
5–Load Reduction
9–Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

SURRY 2

FACILITY DATA

Facility Description

1.	Location (state/countv) (nearest population center)	Virginia/Surry 17 miles NW of Newport News
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2441
5.	Design Electrical Rating (MWE Net)	822
6.	Date of Initial Criticality	March 7, 1973
7.	Date of Commercial Operation	May 1, 1973
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	James River

Utility & Contractor Information

10.	Licensee	Virginia Electric and Power Company
11.	Corporate Address	P. O. Box 26666 Richmond, Virginia 23261
12.	Corporate Contact	J. H. Ferguson Exec. V.P. - Power
13.	Architect/Engineer	Stone & Webster
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R.C. Lewis
18.	IE Section Chief	P. J. Kellog
19.	IE Resident Inspector	D. Burke (804) 357-2101
20.	Licensing Project Manager	D. Neighbors
21.	Docket Number	50-281
22.	License Number/Date of Issue	DPR-37, 1/29/73
23.	Public Document Room	Swem Library College of William and Mary Williamsburg, Virginia 23185

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

SYSTEM MODIFICATIONS AND REANALYSIS FOR SEISMIC SUPPORTS ARE CONTINUING.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

UNIT IS CURRENTLY AT 100% POWER.

♦ LAST IE SITE INSPECTION DATE: SEPTEMBER 28 - OCTOBER 2, 1980

♦ INSPECTION REPORT NO: 50-281/80-42

SUMMARY

INSPECTION SUMMARY

♦ INSPECTION SEPTEMBER 16-19 (80-39): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 36 INSPECTOR-HOURS ON SITE IN THE AREAS OF STARTUP TEST RESULTS REVIEW WHICH INCLUDED THE INTEGRATED LEAK RATE, LOCAL LEAK RATE, THERMAL EXPANSION, AND LOAD REJECTION TESTING. OF THE FOUR AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION SEPTEMBER 28 - OCTOBER 2 (80-42): INCLUDED REVIEW OF THE SECURITY PLAN, PHYSICAL BARRIERS (PROTECTED AREA), PHYSICAL BARRIERS (VITAL AREA), LIGHTING, ACCESS CONTROL (IDENTIFICATION, AUTHORIZATION, AND BADGING), ACCESS CONTROL (SEARCH), ACCESS CONTROL (ESCORTING) AND ACCESS CONTROL (VITAL AREAS). THE INSPECTION INVOLVED 30 INSPECTOR HOURS ON SITE BY ONE NRC INSPECTOR. THE INSPECTION WAS BEGUN DURING AN OFFSHIFT PERIOD; FIVE INSPECTION HOURS WERE ACCOMPLISHED DURING OFFSHIFT PERIODS. THE LICENSEE WAS FOUND TO BE IN COMPLIANCE WITH NRC REQUIREMENTS WITHIN THE EIGHT AREAS EXAMINED DURING THE INSPECTION EXCEPT FOR THE FOLLOWING FOUR ITEMS: (1) ACCESS CONTROL (SEARCH) - NO COMMITMENT IN THE SECURITY PLAN TO A PERCENTAGE HANDS-ON SEARCH OF NON-REGULAR EMPLOYEES; (2) ACCESS CONTROL (SEARCH) - FAILURE TO HAVE COMPENSATORY MEASURES FOR PERSONNEL SEARCH EQUIPMENT; (3) ACCESS CONTROL (VITAL AREAS) - FAILURE TO POSITIVELY CONTROL ALL POINTS OF PERSONNEL ACCESS INTO VITAL AREAS; AND (4) ACCESS CONTROL (VITAL AREAS) - FAILURE TO HAVE ADEQUATE COMPENSATORY PROCEDURES FOR CONTROLLING ACCESS INTO VITAL AREAS.

INSPECTION SEPTEMBER 2 - OCTOBER 3 (80-43): THIS INSPECTION INVOLVED 49 (RESIDENT) INSPECTOR-HOURS ON SITE IN THE AREAS OF PLANT OPERATIONS AND OPERATING RECORDS, PLANT MODIFICATIONS, MAINTENANCE AND TESTING, REFUELING ACTIVITIES, LICENSEE EVENT REPORTS, AND PLANT SECURITY. OF THE SIX AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED IN FOUR AREAS; TWO ITEMS OF NONCOMPLIANCE WERE FOUND IN TWO AREAS (INFRACTION - FAILURE TO PROPERLY EVALUATE AND DOCUMENT INSTALLATION OF A JUMPER HOSE ON SUMP PIPING; INFRACTION - FAILURE TO PROPERLY SEAL UNIT 1 CONTAINMENT DOOR DURING REFUELING).

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

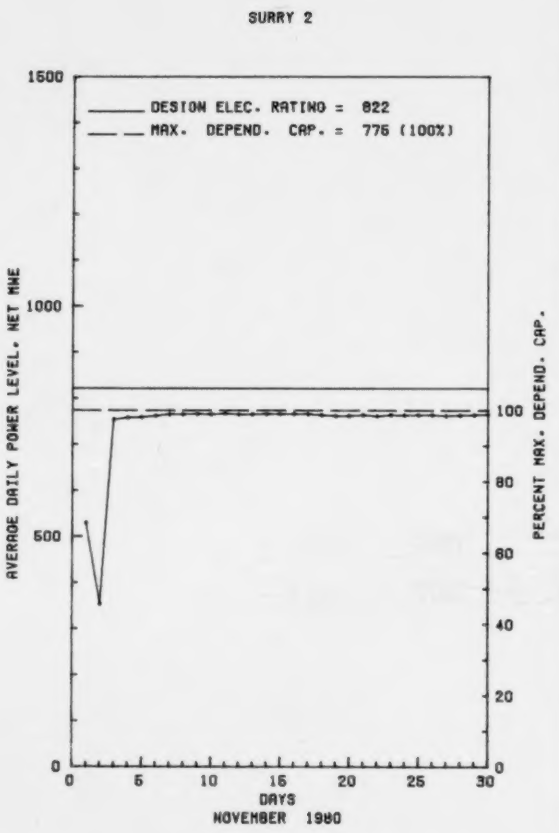
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE.			

REVIEWED BY OMPA LOEB: John L [illegible] DATE: 1/16/81

REVIEWED BY NRR DL: Don Neighbors DATE: 1/16/81

SURRY 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-281
2. Utility Contact: SUE DUNN (804) 357-3184
3. Licensed Thermal Power (MWt): 2441
4. Nameplate Rating (Gross MWe): 942 x 0.9 = 848
5. Design Electrical Rating (Net MWe): 822
6. Maximum Dependable Capacity (Gross MWe): 811
7. Maximum Dependable Capacity (Net MWe): 775
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	8,040.0	66,505.0
12. Hours Reactor Was Critical	711.4	2,543.1	37,040.0
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	705.7	2,396.4	36,392.6
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	1,709,184	5,442,903	84,636,986
17. Gross Electrical Energy (MWH)	563,520	1,773,040	27,641,884
18. Net Electrical Energy (MWH)	535,292	1,675,180	26,211,785
19. Unit Service Factor	98.0	29.8	54.7
20. Unit Availability Factor	98.0	29.8	54.7
21. Unit Capacity Factor (MDC Net)	95.9	26.9	50.9
22. Unit Capacity Factor (DER Net)	90.4	25.3	47.9
23. Unit Forced Outage Rate	2.0	4.0	20.1

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
SNUBBER INSPECTION - MARCH 1981 - 4 DAYS
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-19	11/01/80	F	14.3	G	3	80-035			CONSTRUCTION WORKER GROUNDED ONE PHASE OF 'C' RESERVE STATION SERVICE TRANSFORMER WHILE CHIPPING CEMENT IN TURBINE BUILDING BASEMENT. UNIT TRIPPED ON 'C' S/G HI WATER LEVEL. REPLACED DAMAGED CABLE.

SUMMARY

SURRY 2 OPERATED AT NEAR FULL-POWER FOR NOVEMBER, WITH 1 REDUCTION ON THE 1ST.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

THREE MILE ISLAND 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Pennsylvania/Dauphin 10 miles SE of Harrisburg
2.	Electric Reliability Council	Mid-Atlantic Area Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2535
5.	Design Electrical Rating (MWE Net)	819
6.	Date of Initial Criticality	June 5, 1974
7.	Date of Commercial Operation	September 2, 1974
8.	Condenser Cooling Method	Closed
9.	Condenser Cooling Water Source	Susquehanna River

Utility & Contractor Information

10.	Licensee	Metropolitan Edison Company
11.	Corporate Address	P. O. Box 542 Reading, Pennsylvania
12.	Corporate Contact	R. C. Arnold, Sr. Vice-President 260 Cherry Hill Rd. Parsippany, NJ 07054
13.	Architect/Engineer	Gilbert Associates, Inc.
14.	Nuclear Steam Supply System	B&W
15.	Constructor	UE&C

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Section Chief	A. N. Fasano (717) 948-1123 (FTS) 590-1123
19.	IE RESIDENT INSPECTOR	Haverkamp/Young (717) (717) 948-1127/29 (FTS (FTS) 590-1127/29
20.	Licensing Project Manager	D. C. DiIanni / H. Silver (X27435)
21.	Docket Number	50-289
22.	License Number/Date of Issue	DPR-50, 4/19/74
23.	Public Document Room	Government Publications Section State Library of Pennsylvania Education Building Commonwealth and Walnut Street Harrisburg, Pennsylvania 17126

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

COMPLETE REQUIRED MODIFICATIONS PRIOR TO RESTART.

PLANT STATUS:

THE PLANT REMAINS SHUTDOWN, BY NRC ORDER, PENDING COMPLETION OF MODIFICATIONS AND OTHER ACTIONS RELATED TO THE TMI-2 ACCIDENT.

+ LAST IE SITE INSPECTION DATE: 11/3-30/80

+ INSPECTION REPORT NO: 50-289/80-30

SUMMARY

INSPECTION SUMMARY

+ 50-289/80-22 - JUL 28 - AUG 8: SPECIAL HEALTH PHYSICS EVALUATION, BY FOUR REGION-BASED INSPECTORS AND TWO INDIVIDUALS FROM NRC HEADQUARTERS OFFICES TO EVALUATE THE OVERALL ADEQUACY AND EFFECTIVENESS OF THE IMPLEMENTATION OF THE RADIATION PROTECTION AND EMERGENCY PLANNING PROGRAMS. THE EVALUATION INCLUDED OBSERVATIONS OF WORK PRACTICES, REVIEWS OF SELECTED PROCEDURES AND RECORDS, AND INTERVIEWS WITH GENERAL PUBLIC UTILITIES AND CONTRACTOR PERSONNEL. THE AREAS OF THE EVALUATION INCLUDED: RADIATION PROTECTION ORGANIZATION AND MANAGEMENT; PERSONNEL SELECTION, QUALIFICATIONS AND TRAINING; EXPOSURE CONTROLS (INTERNAL AND EXTERNAL); RADIOACTIVE WASTE MANAGEMENT; ALARA; FACILITIES AND EQUIPMENT; MANAGEMENT OVERSIGHT; EMERGENCY PLAN IMPLEMENTATION; NUREG 0600, CORRECTIVE ACTIONS; AND, NUREG 0578, SHORT TERM LESSONS LEARNED. FIVE NONCOMPLIANCES WERE IDENTIFIED: FAILURE TO DEVELOP AND IMPLEMENT RESPIRATORY PROTECTION MAINTENANCE PROCEDURES; FAILURE TO ADHERE TO RADIATION PROTECTION PROCEDURES; FAILURE TO PERFORM SURVEYS; FAILURE TO IMPLEMENT A QUALITY ASSURANCE PROGRAM; AND FAILURE TO ESTABLISH APPROVED PROCEDURES.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

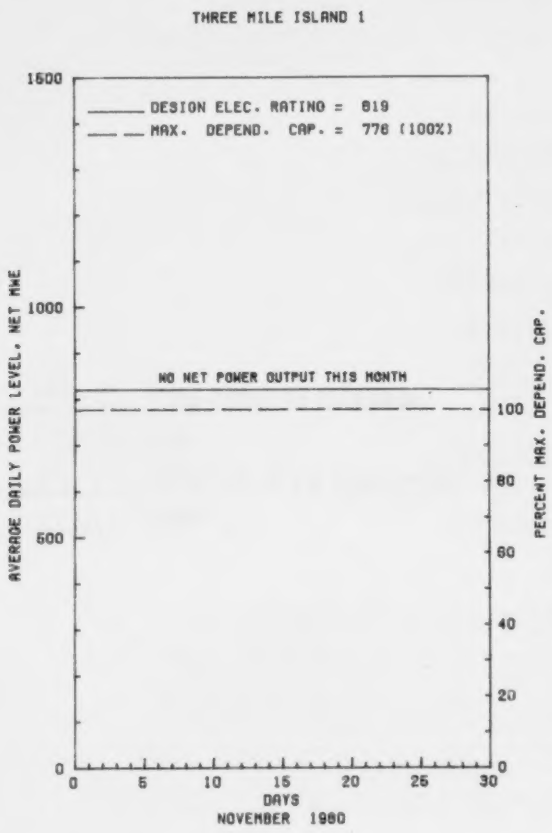
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
NONE			

REVIEWED BY OMPA LOEB: E Bobe **DATE:** 1/16/81

REVIEWED BY NRR DL: D C Di **DATE:** 1/19/81

THREE MILE ISLAND 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-289
2. Utility Contact: D. G. Mitchell (215) 921-6579
3. Licensed Thermal Power (MWt): 2535
4. Nameplate Rating (Gross MWe): 968 x 0.9 = 871
5. Design Electrical Rating (Net MWe): 819
6. Maximum Dependable Capacity (Gross MWe): 840
7. Maximum Dependable Capacity (Net MWe): 776
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

ITEM 21 CALCULATED WITH A WEIGHTED VALUE.

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	8,040.0	54,769.0
12. Hours Reactor Was Critical	.0	.0	31,731.8
13. Reactor Reserve Shutdown Hours	.0	.0	838.5
14. Hours Generator On-Line	.0	.0	31,180.9
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	0	76,531,871
17. Gross Electrical Energy (MWH)	0	0	25,484,330
18. Net Electrical Energy (MWH)	0	0	23,840,053
19. Unit Service Factor	.0	.0	56.9
20. Unit Availability Factor	.0	.0	56.9
*21 Unit Capacity Factor (MDC Net)	.0	.0	55.3
22. Unit Capacity Factor (DER Net)	.0	.0	53.1
23. Unit Forced Outage Rate	100.0	100.0	34.2

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: HEARING IN PROCESS.

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
1	02/17/79	F	720.0	D	4				REGULATORY RESTRAINT ORDER CONTINUES.

SUMMARY

THREE MILE ISLAND 1 REMAINS SHUTDOWN FOLLOWING THE ACCIDENT AT THREE MILE ISLAND 2.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

THREE MILE ISLAND 2

NRC POWER RESTRICTIONS: SEE "LICENSE STATUS"

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Pennsylvania/Dauphin 10 miles SE of Harrisburg
2.	Electric Reliability Council	Mid-Atlantic Area Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2772
5.	Design Electrical Rating (MWE Net)	906
6.	Date of Initial Criticality	3/28/78
7.	Date of Commercial Operation	12/30/78
8.	Condenser Cooling Method	CLOSED
9.	Condenser Cooling Water Source	Susquehanna River

Utility & Contractor Information

10.	Licensee	Metropolitan Edison Company
11.	Corporate Address	P. O. Box 542 Reading, PA 19603
12.	Corporate Contact	R. C. Arnold, Sr. Vice-President 100 Interpace Parkway Parsippany, NJ 07054
13.	Architect/Engineer	Burns & Roe
14.	Nuclear Steam Supply System	B&W
15.	Constructor	UE&C

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Section Chief	A. N. Fasano FTS: 590-1123 (717) 948-1123
19.	IE Resident Inspector	R. Conte FTS: 590-1125 (717) 948-1123
20.	Licensing Project Manager	D. Brinkman
21.	Docket Number	50-320
22.	License Number/Date of Issue	DPR-73/2; 2/8/78
23.	Public Document Room	Government Publications Section State Library of Pennsylvania Education Building Commonwealth & Walnut Streets Harrisburg, Pennsylvania 17126

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEM AND COMPONENT PROBLEMS:

CLEANUP OPERATIONS FROM 3/28/79 ACCIDENT.

+ PLANT STATUS:

+ DECAY HEAT IS BEING REMOVED BY HEAT TRANSFER THROUGH REACTOR COOLANT SYSTEM BOUNDARY TO THE REACTOR BUILDING AMBIENT.

+ LAST IE SITE INSPECTION DATE: 11/17-21/80

+ INSPECTION REPORT NO: 50-320/80-18

SUMMARY

INSPECTION SUMMARY

+ 50-320/80-12 - JUL 6 - AUG 31: ROUTINE INSPECTION BY FIVE RESIDENT INSPECTORS (266 HOURS) AND THREE REGION-BASED INSPECTORS (23 HOURS) OF LICENSEE ACTION ON PREVIOUS INSPECTION FINDINGS; SELECTED NEW AND REVISED PROCEDURES; LICENSEE PROCEDURE IMPLEMTNATION; PLANT OPERATIONS; HEALTH PHYSICS AND ENVIRONMENTAL INSPECTION AND REVIEW; EMERGENCY DRILL; CONSTRUCTION ASPECTS OF SUBMERGED DEMINERALIZER SYSTEM (SDS); PREOPERATIONAL ASPECTS OF MINI DECAY HEAT REMOVAL (MDHR) SYSTEM; AND IN OFFICE REVIEW OF LICENSEE EVENT REPORTS. NO NONCOMPLIANCES WERE IDENTIFIED.

LICENSE STATUS

By Order for Modification of License dated July 20, 1979, the licensee's authority to operate the facility was suspended and the licensee's authority was limited to maintenance of the facility in the present shutdown cooling mode (44 F.R. 45271).

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-022/ 03L	05/18/80	06/17/80	INOPERABLE FHB VENTILATION DUE TO LOW FHB EXHAUST FLOW DUE TO MANUAL INLET DAMPER FAILURE (CLOSED DUE TO UNIVERSAL COUPLING FAILURE - REPAIRED (TS 3.9.12)
80-023/ 03L	06/03/80	07/03/80	HIGH BORON CONC. DUE TO USE OF TEMPORARY AIR SPARGER FOR BAMT MIXING-AIR CAUSED EVAPORATION AND CONCENTRATING EFFECT FOR BORON IN BAMT (TS 3.1.1.1)
80-024/ 03L	05/29/80	06/30/80	RCS PRESSURE INDICATOR WAS ISOLATED FROM RCS DUE TO DEFICIENT PROCEDURE (TS 3.3.3.6)
80-025/ 03L	06/07/80	07/07/80	REACTOR COOLANT INLET TEMPERATURE INDICATOR ON REMOTE SHUT DOWN PANEL FAILED-TEMPORARY VOLTMETER INSTALLED (TS 3.3.3.5)
80-026/ 03L	06/18/80	07/18/80	ONE FIRE PUMP INADVERTENTLY TAKEN OUT OF SERVICE LEAVING 2 OF 4 PUMPS OPERABLE-WHEN IDENTIFIED ANOTHER FIRE PUMP WAS PUT IN SERVICE (TS 3.7.10.1)
80-027/ 01L	07/09/80	08/08/80	RB PURGE EXHAUST FILTER FIRE PROTECTION DELUGE VALVES ISOLATED TO PREVENT INADVERTENT ACTUATION-TECH SPECS TO BE REVISED (TS 3.7.10.2)
80-028/ 03L	07/07/80	08/06/80	EMERGENCY DIESEL TRIPPED DURING SURVEILLANCE TEST DUE TO LOW CRANK CASE PRESSURE WHICH RESULTED FROM A HOSE VIBRATING OFF-APPARENT LOOSE CLAMP (TS 3.8.1.1.B)
80-029/ 01L	07/14/80	08/18/80	BALANCE OF PLANT DIESEL (GRAY) FAILED TO START DUE TO DAMAGED SPEED SENSOR CONNECTOR-DAMAGE APPARENTLY DUE EXPOSURE TO A WALKWAY-CONDUIT WAS RERUN (TS 3.8.1.1.C)
80-030/ 01L	07/17/80	08/18/80	CONTAINMENT PERSONNEL AIRLOCK INNER DOOR SEAL FAILED LEAKAGE TESTS (TS 4.6.1.3.A) DUE TO FOREIGN MATERIAL ON SEAL SURFACE (TS 3.6.1.3.A) RECLEANING AND TESTING WAS SUCCESSFUL
80-031/ 03L	06/23/80	07/23/80	BALANCE OF PLANT DIESEL TRIPPED DURING SURVEILLANCE TESTING FOR UNKNOWN REASON-RETEST WAS SATISFACTORY-LER 80-29 REPORTS DETERMINATION
80-032/ 03L	07/19/80	08/18/80	LEAK IN COUPLING FOR EPICOR I RADIOACTIVE LIQUID CLEANUP SYSTEM DUE WORN COUPLING (TS 6.9.1.9(D))
80-033/ 01L	07/30/80	09/08/80	PNEUMATIC TIMER MALFUNCTION ON EMERGENCY DIESEL GENERATOR APPARENTLY DUE TO END OF LIFE FOR COMPONENT-LICENSEE TO EVALUATE (TS 6.9.1.8.I)
80-034/ 03L	07/25/80	08/25/80	OPERATORS FAILED TO RESTORE FUEL HANDLING BUILDING VENTILATION ON INADVERTENT TRIP DUE TO FALSE FIRE ALARM (TS 3.9.12(A))
80-036/ 01L	08/08/80	09/05/80	REACTOR COOLANT SYSTEM CHEMISTRY ANALYSIS FOR BORON AND HYDROGEN NOT PERFORMED WITHIN REQUIRED TIME PERIOD DUE TO UNAVAILABILITY OF CONTRACTOR LAB - SUBSEQUENT ANALYSIS WAS SAT. (TS 4.1.1.2)
80-035/ 03L	07/31/80	09/08/80	FIRE DETECTOR FOR BALANCE OF PLANT DIESEL (WHITE) FAILED-FIRE WATCH INITIATED UNTIL DETECTOR WAS REPLACED (TS 3.3.3.8) TS TO BE CHANGED TO REFLECT EQUIPMENT NO LONGER REQUIRED
80-037/ 01L	08/08/80	09/09/80	OUTER DOOR SEALS FOR AIRLOCK NO. 2 TO RX BUILDING FAILED LEAK RATE TEST DUE TO APPARENT FAILURE TO PROPERLY OPEN DOOR ON INITIAL OPENING (TS 3.6.1.3)
80-038/ 01L	08/26/80	09/24/80	CANCELLED-ERRONEOUS PROMPT REPORT DUE TO FLOW INDICATOR FALSE READING IN AUXILIARY BUILDING VENTILATION SYSTEM
80-039/ 03L	08/22/80	09/22/80	CABLE/TRANSFORMER HALON SYSTEM FAILED SURVEILLANCE TEST-FULL CHARGE WEIGHT NOT MAINTAINED DUE TO PROCEDURE DEFICIENCY-PROBLEM CORRECTED TS 3.7.10.3
80-041/ 01L	09/09/80	10/09/80	INCORE THERMOCOUPLE M-9 FAILED REASON FOR FAILURE OF THE INCORE INSTRUMENT IS NOT KNOWN-INSTRUMENT INACCESSIBLE FOR REPAIRS (TS 3.3.6 TBL 3.10 NO. 10)
80-042/ 03L	09/09/80	10/09/80	REACTOR COOLANT SYSTEM FLOW RATE INSTRUMENT FAILED CALIBRATION DUE TO INSTRUMENT DRIFT-INSTRUMENT WAS RECALIBRATED (TS 3.3.3.5)
80-043/ 03L	09/02/80	10/02/80	AUXILIARY BUILDING VENTILATION TRAIN B OUT OF SERVICE WHILE TRAIN A WAS NOT IN SERVICE-FAN 8D TRIPPED (IN B TRAIN) DUE TO CIRCUIT BREAKER FAILURE-REPAIRS MADE (TS 3.9.12(A))
80-044/ 01L	09/25/80	10/27/80	REACTOR BUILDING PERSONNEL AIRLOCK NO. 1 OUTER DOOR SEAL FAILED LEAK RATE TEST-IMMEDIATE ACCESS FOR SEAL REPLACEMENT WAS NOT AVAILABLE WITHIN 24 HOURS-SEALS REPLACED AND TESTED (TS 3.6.1.3)
80-045/ 03L	09/25/80	10/29/80	DURING TEST OF DIESEL 1B IT FAILED TO START DUE TO OPERATOR ERROR - INADVERTENT SETTING OF LOAD LIMIT CONTROL TO MINIMUM INSTEAD OF GOVERNOR CONTROL-REINSTRUCTED/PROC. CHANGE (TS 3.8.1.1)
80-046/ 01L	09/30/80	10/30/80	FEEDER BREAKERS FOR BUSSES 2-38 AND 2-48 REMOVED FROM SERVICE FOR MAINTENANCE-EXCEEDED PERMITTED TIME LIMIT TO BE INOPERABLE DUE TO OPERATOR ERROR (TS 3.8.12) TRAINING TO BE DONE
80-047/ 03L	10/16/80	11/17/80	REACTOR BLDG PERSONNEL AIRLOCK NO. 2 INNER DOOR SEAL FAILED LEAK TEST DUE TO APPARENT FOREIGN MATERIAL ON SEALING SURFACE-CLEANED AND SATISFACTORILY PASSED LEAK TESTING (TS 3.6.1.3)
80-048/ 03L	10/17/80	11/17/80	INNER CONTAINMENT ISOLATION VALVE (CF-V115) FOR CFT SAMPLE LINE ISOLATION FAILED TO CLOSE AFTER OPENING-VALVE IS SUBMERGED IN CONTAINMENT AND THEREFORE IS INACCESSIBLE (TS 3.6.1.1)

REVIEWED BY OMPA LOEB: PE Role

DATE: 1/16/81

REVIEWED BY NRR TMIPO: D. Brinkman

DATE: 1/19/81

THREE MILE ISLAND 2

AVERAGE DAILY POWER LEVEL (MWe)

OPERATING STATUS

1. Docket Number: 50-320
2. Utility Data Prepared By: R. A. Lengel (215) 921-6581
3. Licensed Thermal Power (MWt): 2772
4. Nameplate Rating (Gross MWe): 1068 X 0.9 = 961
5. Design Electrical Rating (Net MWe): 906
6. Maximum Dependable Capacity (Gross MWe): 0
7. Maximum Dependable Capacity (Net MWe): 0
8. If Changes Occur in Capacity Ratings (Items Number 3 Through 7) Since Last Report, Give Reasons
None
9. Power Level To Which Restricted, If Any (Net MWe): None
10. Reasons For Restrictions, If Any

This Month Yr

11. Hours In Reporting Period
12. Number Of Hours Reactor Was Critical
13. Reactor Reserve Shutdown Hours
14. Hours Generator On-Line
15. Unit Reserve Sh
16. Gro
(MWH)
1b
20. U Availability Factor
21. Unit Capacity Factor (Using MDC Net) 1/
22. Unit Capacity Factor (Using DER Net) 1/
23. Unit Forced Outage Rate
24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
None
25. If Shut Down At End Of Report Period, Estimated Date of Startup: INDEFINITE

NO DECISION ON FUTURE OPERATION
SHUTDOWN INDEFINITELY

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
SEE SUMMARY BELOW.									

SUMMARY

THE UNIT HAS REMAINED SHUTDOWN SINCE THE 03/28/79 ACCIDENT, WHICH CAUSED EXTENSIVE CORE DAMAGE.

AUTHORITY TO OPERATE FACILITY SUSPENDED BY NRC ORDER DATED JULY 20, 1979, WHICH REQUIRES LICENSEE TO MAINTAIN FACILITY IN A SHUTDOWN CONDITION.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

TROJAN

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1. Location (state/county) (nearest population center): Oregon/Columbia, 42 miles N of Portland
2. Electric Reliability Council: Western Systems Coordinating Council
3. Type of Reactor: PWR
4. License Power Level (MWT): 3411
5. Design Electrical Rating (MWE Net): 1130
6. Date of Initial Criticality: December 15, 1975
7. Date of Commercial Operation: May 20, 1976
8. Condenser Cooling Method: Natural draft cooling tower
9. Condenser Cooling Water Source: Columbia River

Utility & Contractor Information

10. Licensee: Portland General Electric Co.
11. Corporate Address: 121 S. W. Salmon St. Portland, Oregon 97204
12. Corporate Contact: C. Goodwin, Jr., Asst. V.P.
13. Architect/Engineer: Bechtel
14. Nuclear Steam Supply System: Westinghouse
15. Constructor: Multiple

Regulatory Information

16. IE Region responsible: V
17. IE Branch Chief: J. L. Crews
18. IE Section Chief: D. M. Sternberg
19. IE Resident Inspector: M. H. Malmros (503) 556-3751
20. Licensing Project Manager: C. M. Trammell
21. Docket Number: 50-344
22. License Number/Date of Issue: NPF-1, 11/21/75
23. Public Document Room: Columbia County Courthouse, Law Library Circuit Court Room, St. Helens, Oregon 97501

INSPECTION STATUS

ENFORCEMENT STATUS

10 CFR 50.59, "CHANGES, TESTS AND EXPERIMENTS," AUTHORIZES THE LICENSEE TO MAKE CHANGES IN THE FACILITY AND PROCEDURES DESCRIBED IN THE SAFETY ANALYSIS REPORT, AND TO CONDUCT TESTS OR EXPERIMENTS NOT DESCRIBED IN THE SAFETY ANALYSIS REPORT WITHOUT PRIOR COMMISSION APPROVAL, UNLESS THE PROPOSED CHANGE, TEST OR EXPERIMENT INVOLVES A CHANGE IN THE TECHNICAL SPECIFICATIONS INCORPORATED IN THE LICENSE OR AN UNREVIEWED SAFETY QUESTION. THE LICENSEE MUST MAINTAIN A RECORD OF SUCH A CHANGE, TEST OR EXPERIMENT THAT INCLUDES A WRITTEN SAFETY EVALUATION WHICH PROVIDES THE BASIS FOR THE DETERMINATION THAT THE CHANGE, TEST OR EXPERIMENT DOES NOT INVOLVE AN UNREVIEWED SAFETY QUESTION. FINAL SAFETY ANALYSIS REPORT, SECTION 5.1.3.3, STATES IN PART "IF THE RCS IS TO BE OPENED DURING THE SHUTDOWN, THE HYDROGEN AND FISSION GAS IN THE REACTOR COOLANT IS REDUCED BY DEGASSING THE COOLANT IN THE VOLUME CONTROL TANK." CONTRARY TO THE REQUIREMENTS, FROM APRIL 11 TO APRIL 14, 1980 THE REACTOR COOLANT SYSTEM (RCS) WAS DEGASSED BY VENTING THE PRESSURIZER VAPOR SPACE VIA A JUMPER TO THE COOLANT VOLUME CONTROL SYSTEM HOLDUP TANK, AND AN EVALUATION WAS NOT MADE OF THIS CHANGE, TEST OR EXPERIMENT TO DETERMINE THAT IT DID NOT INVOLVE AN UNREVIEWED SAFETY QUESTION.
(8016 INFRACTION)

10 CFR 19.12, "INSTRUCTIONS TO WORKERS," STATES IN PART THAT ALL INDIVIDUALS WORKING IN OR FREQUENTING ANY PORTION OF A RESTRICTED AREA SHALL BE INSTRUCTED IN THE APPROPRIATE RESPONSE TO WARNINGS MADE IN THE EVENT OF ANY UNUSUAL OCCURRENCE OR MALFUNCTION THAT MAY INVOLVE EXPOSURE TO RADIATION OR RADIOACTIVE MATERIAL. CONTRARY TO THE REQUIREMENTS, ON JULY 7, 1980 THREE INDIVIDUALS WERE GRANTED UNESCORTED ACCESS TO PORTIONS OF THE RESTRICTED AREA INCLUDING AREAS POSTED "CAUTION: EVACUATION ALARM OR PAGING SYSTEM CANNOT BE HEARD," AND WERE NOT INSTRUCTED IN THE ADMINISTRATIVE CONTROLS NECESSARY TO PERMIT AN APPROPRIATE RESPONSE TO WARNINGS MADE IN THE EVENT OF ANY UNUSUAL OCCURRENCE OR MALFUNCTION THAT MAY INVOLVE EXPOSURE TO RADIATION OR RADIOACTIVE MATERIAL.
(8016 INFRACTION)

INSPECTION STATUS

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

PURSUANT TO THE ASLB DECISION OF JULY 11, 1980, A LICENSE AMENDMENT WAS ISSUED ON JULY 25, 1980 AUTHORIZING THE LICENSEE TO MODIFY THE CONTROL BUILDING TO RESTORE THE INITIAL DESIGN MARGINS.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS

THE PLANT BEGAN AND ENDED THE MONTH AT FULL POWER. NO SHUTDOWNS OCCURRED THIS MONTH.

LAST IE SITE INSPECTION DATE: 11/03-07/80+

INSPECTION REPORT NO: 50-344/80-28+

REPORTS RECEIVED FROM LICENSEE

NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-21/ 03L-1	10/03/80	11/19/80	FAILURE OF MSIV TO FULLY CLOSE ON TEST.
80-21/ 01T-0	10/03/80	10/31/80	MSIV-C FAILED TO FULLY CLOSE DURING TEST.
80-22/ 01T-0	10/28/80	10/31/80	REPORTING OF UNQUALIFIED ELECTRICAL EQUIPMENT PER IEB 79-01B.
80-23/ 03L-0	10/03/80	10/31/80	FAILURE OF MAIN FEED REGULATOR VALVE-B AFTER A TRIP.
80-24/ 03L-0	10/11/80	11/10/80	BLOWN FUSE IN AUTO START CIRCUIT FOR DIESEL AUXILIARY FEED PUMP.

SUMMARY

INSPECTION SUMMARY:

- INSPECTION ON OCTOBER 20 - 29, 1980 (REPORT NO. 50-344/80-23) AREAS INSPECTED INCLUDED A REVIEW OF: ACCESS CONTROL (IDENTIFICATION, AUTHORIZATION AND BADGING), AND FOLLOWUP ON PREVIOUS INSPECTION FINDINGS. THE INSPECTION INVOLVED 40 INSPECTOR HOURS ONSITE BY TWO NRC INSPECTORS.

  RESULTS: NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

- INSPECTION ON OCTOVER 1 - 31, 1980 (REPORT NO. 50-344/80-26) AREAS INSPECTED: ROUTINE INSPECTIONS OF PLANT OPERATIONS, SURVEILLANCE TESTING, PHYSICAL SECURITY, MAINTENANCE, PROCUREMENT, ENVIRONMENTAL MONITORING, FOLLOWUP ON LICENSEE EVENT REPORTS, HEADQUARTERS' REQUESTS, AND ITEMS OF NONCOMPLIANCE. THE INSPECTION INVOLVED 167 INSPECTOR HOURS ONSITE BY THE NRC RESIDENT INSPECTOR.

  RESULTS: NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

- INSPECTION ON OCTOBER 27 - 31, 1980 (REPORT NO. 50-344/80-27) AREAS INSPECTED: ROUTINE, ANNOUNCED INSPECTION BY A REGIONAL BASED INSPECTOR OF ACTIVITIES PERFORMED IN RESPONSE TO IE BULLETIN 79-01B, "ENVIRONMENTAL QUALIFICATION OF CLASS IE EQUIPMENT." THE INSPECTION INVOLVED 32 INSPECTOR HOURS ONSITE AND AT PGE HEADQUARTERS BY ONE NRC INSPECTOR.

  RESULTS: NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

- INSPECTION ON NOVEMBER 3 - 7, 1980 (REPORT NO. 50-344/80-28) AREAS INSPECTED: ROUTINE, UNANNOUNCED INSPECTION BY REGIONAL BASED INSPECTOR OF FACILITY MODIFICATION ACTIVITIES AND PREVIOUSLY IDENTIFIED FOLLOWUP ITEMS. THE INSPECTION INVOLVED 34 INSPECTOR HOURS ONSITE BY ONE NRC INSPECTOR

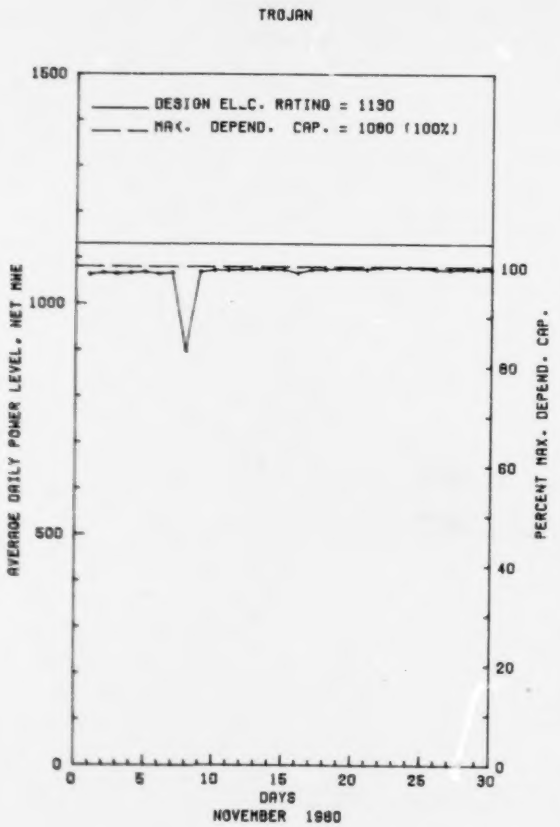
  RESULTS: NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

REVIEWED BY OMPA LOEB: DATE: 1/16/81

REVIEWED BY NRR DL: CM Trammell DATE: 1/19/81

TROJAN

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-344
2. Utility Contact: G. G. Bair (503) 556-3713 x234
3. Licensed Thermal Power (MWt): 3411
4. Nameplate Rating (Gross MWe): 1280 x 0.95 = 1216
5. Design Electrical Rating (Net MWe): 1130
6. Maximum Dependable Capacity (Gross MWe): 1122
7. Maximum Dependable Capacity (Net MWe): 1080
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	8,040.0	37,248.0
12. Hours Reactor Was Critical	720.0	5,719.3	22,017.3
13. Reactor Reserve Shutdown Hours	0	0	2,171.8
14. Hours Generator On-Line	720.0	5,621.8	21,249.6
15. Unit Reserve Shutdown Hours	0	0	1,508.7
16. Gross Thermal Energy (MWH)	2,444,274	17,353,353	66,288,963
17. Gross Electrical Energy (MWH)	804,080	5,645,111	21,630,576
18. Net Electrical Energy (MWH)	768,218	5,365,619	20,398,938
19. Unit Service Factor	100.0	69.9	57.0
20. Unit Availability Factor	100.0	69.9	61.1
21. Unit Capacity Factor (MDC Net)	98.8	61.8	50.7
22. Unit Capacity Factor (DER Net)	94.4	59.1	48.5
23. Unit Forced Outage Rate	0	10.8	26.3

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUELING, APRIL 1981, 45 DAYS
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
80-10	11/08/80	S	.0	B	5				THE POWER WAS REDUCED FROM 100% TO 75% POWER TO ALLOW THE PERFORMANCE OF THE INCORE/EXCORE CALIBRATION PHYSICS TESTING. THE POWER WAS REDUCED FOR 16 HOURS.

SUMMARY

THE PLANT OPERATED THE ENTIRE MONTH AT OR NEAR FULL POWER. A POWER REDUCTION TO 75% OCCURRED ON 11/8 FOR 16 HOURS. STEAM GENERATOR PRIMARY-TO-SECONDARY TUBE LEAKAGE INCREASED FROM 20 GPD AT THE START OF THE MONTH TO ABOUT 45 GPD AT THE END OF THE MONTH.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

TURKEY POINT 3

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Florida/Dade 25 miles S of Miami
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2200
5.	Design Electrical Rating (MWE Net)	693
6.	Date of Initial Criticality	October 20, 1972
7.	Date of Commercial Operation	December 14, 1972
8.	Condenser Cooling Method	Recirculating Canal System
9.	Condenser Cooling Water Source	Canal System

Utility & Contractor Information

10.	Licensee	Florida Power and Light Company
11.	Corporate Address	9250 West Flagler Street P. O. Box 013100 Miami, Florida 33101
12.	Corporate Contact	R. E. Uhrig, V.P. Nuclear Affairs
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. Lewis
18.	IE Section Chief	R. D. Martin
19.	IE Resident Inspector	A. J. Ignatonis (305) 245-7669
20.	Licensing Project Manager	M. Grotenhuis (301) 492-7128
21.	Docket Number	50-250
22.	License Number/Date of Issue	DPR-31, 7/19/72
23.	Public Document Room	Environmental and Urban Affairs Library Florida International University Miami, Florida 33199

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:
NORMAL OPERATION AT POWER. AT THE END OF MONTH, THE UNIT WAS REMOVED FROM SERVICE TO REPAIR LEAKING TUBE PLUG IN STEAM GENERATOR NO. 3.

- LAST IE SITE INSPECTION DATE: NOVEMBER 1-31, 1980
- INSPECTION REPORT NO: 50-250/80-36

SUMMARY

INSPECTION SUMMARY

- INSPECTION OCTOBER 21-24 (80-29): THIS SPECIAL, ANNOUNCED INSPECTION INVOLVED 12 INSPECTOR-HOURS ON SITE IN THE AREAS OF FOLLOWUP ON IE BULLETIN 80-11 AND THE CONTAINMENT BUILDING TENDON SURVEILLANCE INSPECTION PROGRAM. OF THE TWO AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION SEPTEMBER 25-26 (80-30): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 3 INSPECTOR-HOURS ON SITE AND 3 INSPECTOR-HOURS OFF SITE IN THE AREAS OF EMERGENCY PLANNING DRILL OBSERVATION AND REVIEW OF PREVIOUSLY IDENTIFIED UNRESOLVED ITEMS. OF THE TWO AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION OCTOBER 8-10 (80-32): THIS SPECIAL, ANNOUNCED INSPECTION INVOLVED 16 INSPECTOR-HOURS ON SITE IN THE AREAS OF INSTALLED EQUIPMENT REVIEW WITH RESPECT TO IE BULLETIN 79-01B. OF THE AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATION WERE IDENTIFIED.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

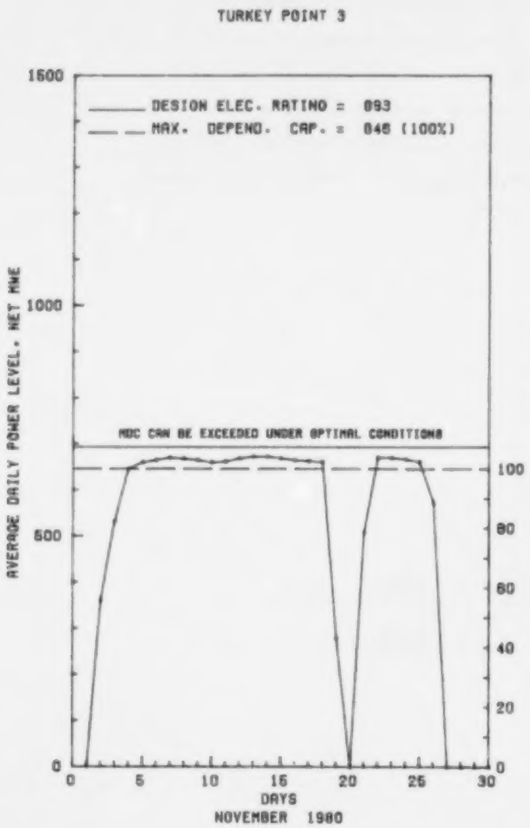
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-017/03L-0	09/18/80	10/20/80	POWER RANGE INDICATION N43 CHANNEL INDICATED LOW, PLACED IN TRIPPED STATE. CAUSE PARTIAL GROUND IN CABLE
80-019/03L-0	10/08/80	10/22/80	RADIOGRAPHS AND ULTRASONIC TEST INDICATED CRACKING IN FEEDWATER NOZZLES IN 3A, 3B AND 3C FEEDWATER LINES
80-020/03L-0	10/18/80	11/14/80	THE PERIODIC TEST OF THE 3A RESIDUAL HEAT REMOVAL PUMP WAS PERFORMED LATE

REVIEWED BY OMPA LOEB: **DATE:** 1/16/81

REVIEWED BY NRR DL: **DATE:** 1/16/81

TURKEY POINT 3

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-250
2. Utility Contact: V. T. Chilson (305) 552-3824
3. Licensed Thermal Power (MWt): 2200
4. Nameplate Rating (Gross MWe): 894 x 0.85 = 760
5. Design Electrical Rating (Net MWe): 693
6. Maximum Dependable Capacity (Gross MWe): 680
7. Maximum Dependable Capacity (Net MWe): 646
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:

NONE

9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:

NONE

ITEM 21 CALCULATED BY USING A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	8,040.0	70,951.6
12. Hours Reactor Was Critical	602.2	6,473.2	53,692.6
13. Reactor Reserve Shutdown Hours	.0	.0	213.3
14. Hours Generator On-Line	561.7	6,210.4	51,909.6
15. Unit Reserve Shutdown Hours	.0	.0	121.8
16. Gross Thermal Energy (MWH)	1,192,309	13,418,298	104,886,432
17. Gross Electrical Energy (MWH)	376,900	4,221,258	33,302,259
18. Net Electrical Energy (MWH)	356,176	3,997,972	31,570,066
19. Unit Service Factor	78.0	77.2	74.1
20. Unit Availability Factor	78.0	77.2	74.3
21. Unit Capacity Factor (MDC Net)	76.6	77.0	70.0
22. Unit Capacity Factor (DER Net)	71.4	71.8	66.9
23. Unit Forced Outage Rate	6.5	1.7	2.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):

REFUELING, MAINT. & INSPECTION - 3/1/81 - 5/9/81

25. If Shut Down At End of Report Period, Estimated Date of Startup: 12/06/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
19	10/06/80	S	21.6	B	4		HB	HTEXCH	UNIT WAS REMOVED FROM SERVICE FOR SCHEDULED STEAM GENERATOR TUBE INSPECTION PROGRAM, MAINTENANCE, AND INSPECTIONS. (CONTINUED FROM PREVIOUS MONTH).
20	11/02/80	S	.6	B	9		HA	MECFUN	UNIT WAS REMOVED FROM SERVICE TO PERFORM SURVEILLANCE TEST ON TURBINE OVERSPEED TRIP DEVICE.
21	11/19/80	F	2.4	A	3		HH	VALVEX	UNIT WAS TRIPPED BY STEAM GENERATOR NO. 3A LEVEL PROTECTION SYSTEM CAUSED BY SUDDEN CLOSURE OF FEEDWATER CONTROL VALVE NO. 3A. CORRECTIVE ACTIONS INCLUDED REPAIRING A LOOSE ELECTRICAL CONNECTION IN THE FEEDWATER VALVE CONTROL SYSTEM.
22	11/19/80	F	19.7	A	3		HH	VALVEX	UNIT WAS TRIPPED BY STEAM GENERATOR NO. 3C LEVEL PROTECTION SYSTEM WHILE REDUCING LOAD TO REMOVE THE UNIT FROM SERVICE TO INVESTIGATE FEEDWATER FLOW CONTROL PROBLEM. OUTAGE WAS EXTENDED TO REPAIR FAILED WELD ON LINE ATTACHED TO BYPASS FEEDWATER LINE TO STEAM GENERATOR NO. 3B.
23	11/20/80	F	16.9	A	9		HH	VALVEX	UNIT WAS REMOVED FROM SERVICE WHEN FEEDWATER FLOW TO STEAM GENERATOR NO. 3A COULD NOT BE TRANSFERRED FROM THE BYPASS FEEDWATER CONTROL VALVE TO THE MAIN FEEDWATER CONTROL VALVE AFTER THE UNIT WAS RETURNED TO SERVICE. INADEQUATE FLOW THROUGH FEEDWATER CONTROL VALVE NO. 3A WAS DUE TO BROKEN VALVE STEM. CORRECTIVE ACTIONS INCLUDED INSTALLING NEW VALVE STEM, PLUG, AND CAGE IN FEEDWATER CONTROL VALVE NO. 3A.
24	11/26/80	S	97.1	B	1		HB	HTEXCH	UNIT WAS REMOVED FROM SERVICE TO LOCATE AND REPAIR LEAKING TUBE PLUG ON STEAM GENERATOR NO. 3B. CORRECTIVE ACTIONS INCLUDED REPAIRING LEAKING TUBE PLUGS BY WELDING.

SUMMARY

UNIT NO. 3 OPERATED AT APPROXIMATELY 100% POWER UNTIL NOV. 26 EXCEPT FOR 4 OUTAGES. ON NOV. 26, THE UNIT WAS REMOVED FROM SERVICE TO LOCATE AND REPAIR A LEAKING TUBE PLUG IN NO. 3 STEAM GENERATOR.

LEGEND

[1] Type:
- F--Forced
- S--Scheduled

[2] Reason:
- A--Equipment Failure
- B--Maintenance or Test
- C--Refueling
- D--Regulatory Restriction
- E--Operator Training & License Examination
- F--Administrative
- G--Operational Error
- H--Other

[3] Method:
- 1--Manual
- 2--Manual Scram
- 3--Automatic Scram
- 4--Continuations
- 5--Load Reduction
- 9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

TURKEY POINT 4

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Florida/Dade 25 miles S of Miami
2.	Electric Reliability Council	Southeastern Electric Reliability Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	2200
5.	Design Electrical Rating (MWE Net)	693
6.	Date of Initial Criticality	June 11, 1973
7.	Date of Commercial Operation	September 7, 1973
8.	Condenser Cooling Method	Recirculating Canal System
9.	Condenser Cooling Water Source	Canal System

Utility & Contractor Information

10.	Licensee	Florida Power and Light Company
11.	Corporate Address	9250 West Flagler Street P. O. Box 013100 Miami, Florida 33101
12.	Corporate Contact	R. E. Uhrig, V. P. Nuclear Affairs
13.	Architect/Engineer	Bechtel
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Bechtel

Regulatory Information

16.	IE Region responsible	II
17.	IE Branch Chief	R. C. Lewis
18.	IE Section Chief	R. D. Martin
19.	IE Resident Inspector	A. J. IGNATONIS (305) 245-7669
20.	Licensing Project Manager	M. Grotenhuis (301) 492-7128
21.	Docket Number	50-251
22.	License Number/Date of Issue	DPR-41, 4/10/73
23.	Public Document Room	Environmental and Urban Affairs Library Florida International University Miami, Florida 33199

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEMS AND COMPONENT PROBLEMS:

NONE.

FACILITY ITEMS (PLANS AND PROCEDURES):

NONE.

MANAGERIAL ITEMS:

NONE.

PLANT STATUS:

REFUELING.

+ LAST IE SITE INSPECTION DATE: NOVEMBER 1-31, 1980

+ INSPECTION REPORT NO: 50-251/80-34

SUMMARY

INSPECTION SUMMARY

+ INSPECTION OCTOBER 21-24 (80-28): THIS SPECIAL, ANNOUNCED INSPECTION INVOLVED 13 INSPECTOR-HOURS ON SITE IN THE AREAS OF FOLLOWUP ON IE BULLETIN 80-11 AND THE CONTAINMENT BUILDING TENDON SURVEILLANCE INSPECTION PROGRAM. OF THE TWO AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

INSPECTION SEPTEMBER 25-26 (80-30): THIS ROUTINE, UNANNOUNCED INSPECTION INVOLVED 3 INSPECTOR-HOURS ON SITE AND 3 INSPECTOR-HOURS OFF SITE IN THE AREAS OF EMERGENCY PLANNING DRILL OBSERVATION AND REVIEW OF PREVIOUSLY IDENTIFIED UNRESOLVED ITEMS. OF THE TWO AREAS INSPECTED, NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

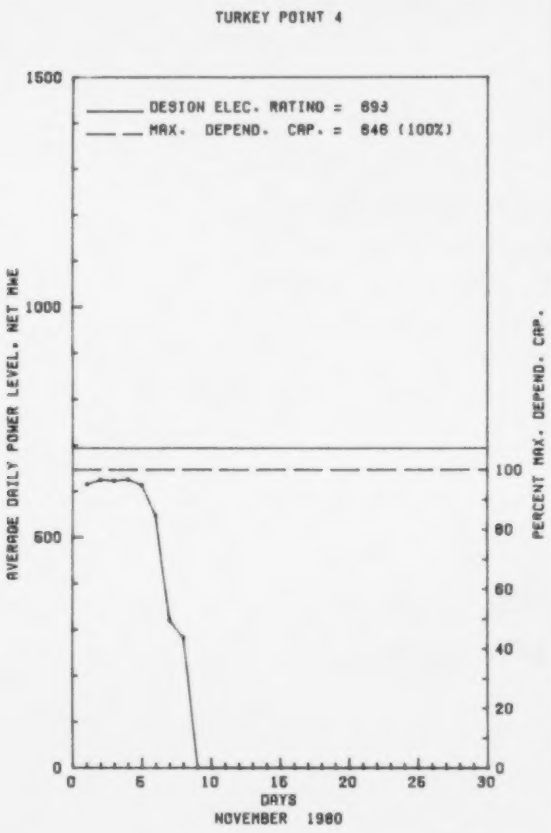
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-018/ 03L-0	09/24/80	10/23/80	CHARGING PUMP SERVICE WATER PUMP WOULD NOT DEVELOP PROPER DISCHARGE HEAD
80-023/ 03L-0	10/03/80	10/31/80	MOV-2867D DECLARED INOPERABLE DUE TO HEAT TRACING INOPERABLE ON BIT
80-026/ 03L-0	10/05/80	10/31/80	OPERATOR FAILED TO RETEST OPERABLE LOW HEAD SAFETY INJECTION PUMP
80-027/ 03L-0	10/05/80	10/31/80	PT-27B REVEALED HEAT TRACING CIRCUIT 6A BELOW ACCEPTANCE CRITERIA
80-028/ 03L-0	09/26/80	10/24/80	REDUCTION SERVICE WATER PRESSURE DUE TO MISSING CHECK VALVE DISC
80-029/ 03L-0	10/12/80	11/07/80	SERVICE WATER PUMP 2-SW-P-10A LOW DISCHARGE PRESSURE DUE TO EEL LODGED IN IMPELLER
80-030/ 03L-0	10/13/80	11/04/80	PT-27B INDICATED LOW AMP ON CIRCUIT 6A HEAT TRACING BORIC ACID BLENDER

REVIEWED BY OMPA LOEB: DATE: 1/16/81

REVIEWED BY NRR DL: DATE: 1/16/81

TURKEY POINT 4

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-251
2. Utility Contact: V. T. Chilson (305) 552-3824
3. Licensed Thermal Power (MWt): 2200
4. Nameplate Rating (Gross MWe): 894 x 0.85 = 760
5. Design Electrical Rating (Net MWe): 693
6. Maximum Dependable Capacity (Gross MWe): 680
7. Maximum Dependable Capacity (Net MWe): 646
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

ITEM 21 CALCULATED BY USING A WEIGHTED AVERAGE.

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	8,040.6	63,769.0
12. Hours Reactor Was Critical	190.1	6,185.1	47,072.9
13. Reactor Reserve Shutdown Hours	.0	12.3	166.6
14. Hours Generator On-Line	190.1	6,096.6	45,279.1
15. Unit Reserve Shutdown Hours	.0	12.3	31.2
16. Gross Thermal Energy (MWH)	372,238	13,162,375	94,381,355
17. Gross Electrical Energy (MWH)	108,135	4,069,695	29,973,173
18. Net Electrical Energy (MWH)	100,462	3,855,365	28,385,361
19. Unit Service Factor	26.4	75.8	71.0
20. Unit Availability Factor	26.4	76.0	71.1
*21. Unit Capacity Factor (MDC Net)	21.6	74.2	69.1
22. Unit Capacity Factor (DER Net)	20.1	69.2	64.2
23. Unit Forced Outage Rate	.0	3	2.9

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 01/10/81

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
20	11/06/80	F	.0	A	5		HA	PIPEXX	LOAD REDUCTION DUE TO FAILED EXPANSION JOINT LOCATED INSIDE THE CONDENSER ON THE EXTRACTION STEAM LINE TO FEEDWATER HEATER NO. 4B. CORRECTIVE ACTIONS INCLUDED REPLACING FAILED EXPANSION JOINTS DURING REFUELING OUTAGE.
21	11/08/80	S	530.0	C	1		RC	FUELXX	UNIT NO 4 WAS REMOVED FROM SERVICE FOR SCHEDULED REFUELING, MAINTENANCE, AND INSPECTIONS.

SUMMARY

THE UNIT WAS REMOVED FROM SERVICE FOR A SCHEDULED REFUELING, MAINTENANCE AND INSPECTION OUTAGE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] **Exhibit F:**
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] **Exhibit H:**
Same source.

VERMONT YANKEE

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Vermont/Windham 5 miles S of Brattleboro
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	BWR
4.	Power Level (MWT)	1593
5.	Design Electrical Rating (MWE Net)	514
6.	Date of Initial Criticality	March 24, 1972
7.	Date of Commercial Operation	November 30, 1972
8.	Condenser Cooling Method	Closed Cycle Cooling Towers
9.	Condenser Cooling Water Source	Connecticut River

Utility & Contractor Information

10.	Licensee	Vermont Yankee Nuclear Power Corp.
11.	Corporate Address	20 Turnpike Road Westboro, Massachusetts 01581
12.	Corporate Contact	D. W. Edwards Assistant to the Vice President
13.	Architect/Engineer	Ebasco
14.	Nuclear Steam Supply System	GE
15.	Constructor	Ebasco

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. Brunner
18.	IE Section Chief	T. T. MARTIN
19.	IE Resident Inspector	W. J. RAYMOND (802) 257-4310
20.	Licensing Project Manager	V. L. Rooney
21.	Docket Number	50-271
22.	License Number/Date of Issue	DPR-28, 3/21/72
23.	Public Document Room	Brooks Memorial Library 224 Main Street Brattleboro, Vermont 05301

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

SYSTEM AND COMPONENT PROBLEMS:

- INSPECTION OF THE REACTOR WATER CLEANUP (RWCU) SYSTEM WELDS IDENTIFIED ADDITIONAL DEFECTS IN THE 4 INCH DIAMETER PIPING ON THE SUCTION SIDE OF THE RWCU PUMPS INSIDE PRIMARY CONTAINMENT. APPROXIMATELY 70 FEET OF SCHEDULE 80 PIPING WILL BE REPLACED. FURTHER EVALUATION OF THE SWEEPOLET, WHICH JOINS THE 4 INCH RWCU LINE TO THE 20 INCH RHR LINE, INDICATED THAT THE SWEEPOLIT MUST BE REPLACED. THE REACTOR CORE WAS COMPLETELY OFF LOADED TO FACILITATE REPAIR OF THE NONISOLABLE PIPE SECTION.
- CHEMTROL CORPORATION ALLEGED THAT FIRE PENETRATION SEALS INSTALLED BY CHEMTROL OF VERMONT YANKEE WERE NOT ADEQUATE.
- CORE SPRAY SPARGER UPPER C JUNCTION BOX REPAIRS WERE COMPLETED.
- JET PUMP BEAM REPLACEMENT WAS ALSO COMPLETED.
- FACILITY ITEMS (PLANS AND PROCEDURES):
- REPAIRS TO THE RWCU SYSTEM ARE IN PPOGRESS WITH CORE LOAD AND COMPLETED OF THE REFUELING OUTAGE PRESENTLY SCHEDULED FOR LATE DECEMBER 1980.
- MANAGERIAL ITEMS:
- PUBLIC INTEREST HAS BEEN EXPRESSED IN AREAS OF REFUELING OUTAGE REPAIRS TO THE RWCU SYSTEM AND THE REQUIRED IE BULLETIN 79-02 ANCHOR BOLT REPLACEMENT AT VERMONT YANKEE.

PLANT STATUS:

- THE REFUELING OUTAGE COMMENCED ON SEPTEMBER 26, 1980. THE CORE HAS BEEN OFF LOADED AND THE REACTOR VESSEL DRAINED TO FACILITATE RWCU SYSTEM REPAIRS.
- LAST IE SITE INSPECTION DATE: 11/19-21/80
- INSPECTION REPORT NO: 50-271/80-20

SUMMARY

INSPECTION SUMMARY

- NONE

REPORTS RECEIVED FROM LICENSEE

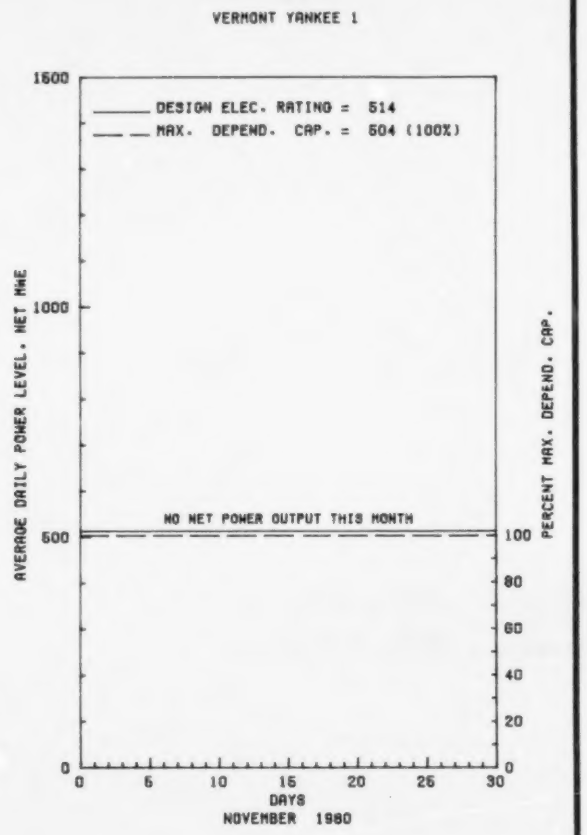
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-030/ 03L	09/29/80	10/29/80	POSITION SWITCH ON ROD 38-35 AND ROD BLOCK ASSOCIATED WITH "ONE CONTROL ROD WITHDRAWAL PERMIT CIRCUIT" WAS FOUND TO BE INOPERABLE
80-031/ 03L	10/01/80	10/31/80	SHOCK SUPPRESSOR MS-24 MAIN STEAM "C" PIPING WAS FOUND WITH NO VISIBLE FLUID IN ITS SIGHT GLASS AND SUBSEQUENTLY FAILED TO LOCK UP WITHIN PROCEDURAL TOLERANCES
80-032/ 01T	10/14/80	10/28/80	CRACK IN C CORE SPRAY SPARGER JUNCTION BOX
80-033/ 01T	10/14/80	10/28/80	CRACK IN #8 JET PUMP HOLD DOWN BEAM
80-034/ 03L	10/13/80	11/12/80	DURING SURVEILLANCE TEST DATA REVIEW IT WAS DISCOVERED THAT THE MONTHLY INTERVAL WAS EXCEEDED BY FOUR DAYS FOR THE OPERABILITY TEST OF THE PLANT FIRE PUMPS
80-035/ 03L	10/07/80	11/06/80	MAIN STEAM RELIEF VALVES HAD AN AS-FOUND SETPOINT OF 1093 PSIG AND SHOULD BE 1090 PSIG
80-036/ 03L	10/17/80	11/14/80	SLC RELIEF VALVES-THE SET-POINTS WERE FOUND TO BE ABOVE TECH SPEC 4.4.A REQUIRED SET-POINT OF 1400 TO 1490 PSIG. SR-39A SETPOINT WAS 1550 PSIG WHILE SR-39B WAS 1600
80-037/ 01T	10/27/80	11/10/80	A LINEAR INDICATION WAS DISCOVERED BY ULTRASONIC TESTING AT WELD F3B ON REACTOR CLEANUP WATER SUCTION LINE CUW-18
80-038/ 03L	10/27/80	11/26/80	LIS-2-3-72A WAS ISOLATED FROM SERVICE DURING INSTALLATION OF THE ANALOG TRIP WITHOUT FIRST FAILING THE TRIP SYSTEM AS REQUIRED BY T/S TABLE 3.2.1
80-039/ 03L	10/25/80	11/24/80	WHILE PERFORMING THE TYPE C LEAK RATE TESTING TWO VALVES EV16-20-20 AND LRW-837 WERE FOUND TO HAVE SEAT LEAKAGE IN EXCESS OF THAT PERMITTED BY T/S SECTION 4.7.A.4

REVIEWED BY OMPA LOEB: John J Rizzo **DATE:** 1-16-81

REVIEWED BY NRR DL: **DATE:** 1/18/81

VERMONT YANKEE

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-271
2. Utility Contact: F.C. BEERS (617) 366-9011
3. Licensed Thermal Power (MWt): 1593
4. Nameplate Rating (Gross MWe): 626 x 0.9 = 563
5. Design Electrical Rating (Net MWe): 514
6. Maximum Dependable Capacity (Gross MWe): 535
7. Maximum Dependable Capacity (Net MWe): 504
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any:
NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	8,040.0	71,834.8
12. Hours Reactor Was Critical	.0	6,266.6	57,425.2
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	.0	6,200.1	55,527.9
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	0	9,347,970	78,572,581
17. Gross Electrical Energy (MWH)	0	3,136,006	26,105,676
18. Net Electrical Energy (MWH)	0	2,972,809	24,741,173
19. Unit Service Factor	.0	77.1	77.3
20. Unit Availability Factor	.0	77.1	77.3
21. Unit Capacity Factor (MDC Net)	.0	73.4	68.3
22. Unit Capacity Factor (DER Net)	.0	71.9	67.0
23. Unit Forced Outage Rate	100.0	4.3	6.3

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: 12/31/80

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type^1	Duration (Hours)	Reason^2	Method of Shutting Down Reactor^3	Licensee Event Report No.	System Code^4	Component Code^5	Cause & Corrective Action to Prevent Recurrence
80-21	09/26/80	S	558.0	C	4		RC	FUELXX	REFUELING CONTINUES.
80-22	11/24/80	F	162.0	A	9	80-37/1T	CG	PIPEXX	THE PIPE CRACKING IS BELIEVED TO HAVE BEEN CAUSED BY IGSCC. THE SWEEP-O-LET IS UNDERGOING A FAILURE ANALYSIS TO DETERMINE THE CAUSE OF THE CRACKING. THE SWEEP-O-LET AND PIPING ARE BEING REPLACED WITH MATERIALS WHICH MEET THE INTENT OF SECTION II OF NUREG 0313, REV.1, DATED OCTOBER, 1979.

SUMMARY

THE UNIT CONTINUED IN A REFUELING AND MAINTENANCE OUTAGE. A PORTION OF THE OUTAGE WAS CHARGED TO THE REPAIR OF A PIPE CRACK.

LEGEND

^{1}Type:
F--Forced
S--Scheduled

^{2}Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

^{3}Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

^{4}Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

^{5}Exhibit H:
Same source.

YANKEE ROWE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Massachusetts/Franklin 25 miles NE of Pittsfield
2.	Electric Reliability Council	Northeast Power Coordinating Council
3.	Type of Reactor	PWR
4.	Power Level (MWT)	600
5.	Design Electrical Rating (MWE Net)	175
6.	Date of Initial Criticality	August 19, 1960
7.	Date of Commercial Operation	July 1961
8.	Condenser Cooling Method	Once Through
9.	Condenser Cooling Water Source	Deerfield River

Utility & Contractor Information

10.	Licensee	Yankee Atomic Electric Company
11.	Corporate Address	20 Turnpike Road Westboro, Massachusetts 01581
12.	Corporate Contact	D. W. Edwards, Asst. to V.P.
13.	Architect/Engineer	Stone & Webster
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Stone & Webster

Regulatory Information

16.	IE Region responsible	I
17.	IE Branch Chief	E. J. Brunner
18.	IE Section Chief	T. T. MARTIN
19.	IE Resident Inspector	T. Foley (413) 424-5242
20.	Licensing Project Manager	R. Caruso (X27232)
21.	Docket Number	50-29
22.	License Number/Date of Issue	DPR-3, 12/24/63
23.	Public Document Room	Greenfield Community College 1 College Drive Greenfield, Massachusetts 01301

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

```
INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS:

+    ROUTINE OPERATIONS.  REACTOR OPERATING AT 100
     PERCENT THERMAL POWER WITH THE TURBINE GENERATOR
     PRODUCTING ABOUT 86 PERCENT ELECTRICAL POWER.

+ LAST IE SITE INSPECTION DATE:  11/1-30/80

+          INSPECTION REPORT NO:  50-29/80-18

     SUMMARY

     INSPECTION SUMMARY

+    NONE
```

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

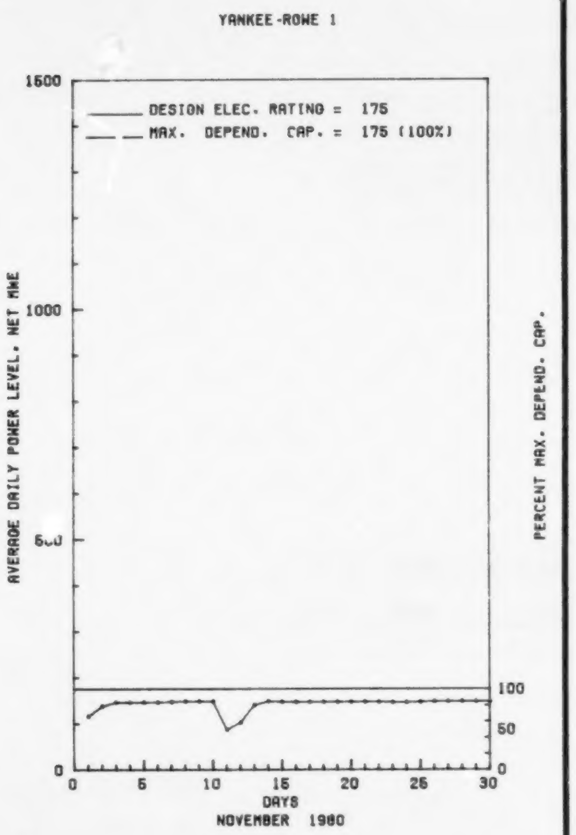
```
==========================================================================================
 NUMBER    DATE OF    DATE OF    SUBJECT
            EVENT      REPORT
==========================================================================================
 NONE
==========================================================================================
```

REVIEWED BY OMPA LOEB: [illegible] **DATE:** 1/16/81

REVIEWED BY NRR DL: R Caruso **DATE:** 1/16/81

YANKEE ROWE

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-029
2. Utility Contact: F.C. BEERS (617) 366-9011
3. Licensed Thermal Power (MWt): 600
4. Nameplate Rating (Gross MWe): 185 x 1.0 = 185
5. Design Electrical Rating (Net MWe): 175
6. Maximum Dependable Capacity (Gross MWe): 180
7. Maximum Dependable Capacity (Net MWe): 175
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons:
NONE
9. Power Level To Which Restricted, If Any (Net MWe): 150
10. Reasons for Restrictions, If Any:
TURBINE BAFFLE PLATE INSTALLATION

CAPACITY FACTORS CALCULATED WITH A WEIGHTED AVERAGE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	8,040.0	175,677.0
12. Hours Reactor Was Critical	716.0	1,225.6	139,590.2
13. Reactor Reserve Shutdown Hours	.0	.0	.0
14. Hours Generator On-Line	710.2	1,190.1	135,217.5
15. Unit Reserve Shutdown Hours	.0	.0	.0
16. Gross Thermal Energy (MWH)	416,361	681,760	72,430,062
17. Gross Electrical Energy (MWH)	110,206	193,522	22,265,675
18. Net Electrical Energy (MWH)	102,544	180,600	20,846,744
19. Unit Service Factor	98.6	14.8	77.0
20. Unit Availability Factor	98.6	14.8	77.0
21. Unit Capacity Factor (MDC Net)	81.4	12.8	70.0
22. Unit Capacity Factor (DER Net)	81.4	12.8	70.0
23. Unit Forced Outage Rate	1.4	82.2	5.4

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each):
NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
3	11/11/80	F	9.8	H	3	80-21	EA	CKTBRK	GROUND TO OFFSITE 115,000 VOLT LINE RESULTING IN LOSS OF Z-126 LINE.

SUMMARY

YANKEE - ROWE 1 OPERATED UNDER SELF-IMPOSED RESTRICTION DURING NOVEMBER, WITH 1 OUTAGE ON THE 11TH DUE TO LOSS OF Z-126 LINE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F:
Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H:
Same source.

ZION 1

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Illinois/Lake 40 miles N of Chicago
2.	Electric Reliability Council	Mid-America Interpool Network
3.	Type of Reactor	PWR
4.	Power Level (MWT)	3250
5.	Design Electrical Rating (MWE Net)	1040
6.	Date of Initial Criticality	June 19, 1973
7.	Date of Commercial Operation	December 31, 1973
8.	Condenser Cooling Method	Straight Through
9.	Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10.	Licensee	Commonwealth Edison Co.
11.	Corporate Address	P. O. Box 767 Chicago, Illinois
12.	Corporate Contact	J. S. Abel, Director of Nuclear Licensing
13.	Architect/Engineer	Sargent & Lundy
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Various

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	R. L. Spessard
19.	IE Resident Inspector	J. E. Kohler (312) 746-2313
20.	Licensing Project Manager	D. Wigginton
21.	Docket Number	50-295
22.	License Number/Date of Issue	DPR-39 issued 4/6/73
23.	Public Document Room	Zion-Benton Public Library 2600 Emmans Avenue Zion, Illinois 60099

INSPECTION STATUS

ENFORCEMENT STATUS

TECHNICAL SPECIFICATION 4.8.8.A.2 REQUIRES THAT IF ONE OF THE TWO HYDROGEN PURGE FAN SYSTEMS IS INOPERABLE, THE REMAINING SYSTEM BE DEMONSTRATED TO BE OPERABLE IMMEDIATELY AND DAILY THEREAFTER. CONTRARY TO THE ABOVE, ON AUGUST 15, 1980, HYDROGEN PURGE FAN 1RV021 WAS MADE INOPERABLE AND THE REQUIRED SURVEILLANCE ON HYDROGEN PURGE FAN 1RV020 WAS NOT PERFORMED. THIS EVENT WAS REPORTED BY THE LICENSEE IN LER 50-295/80-07.
(8017 DEFICIENCY)

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

UNIT 1 OPERATING ROUTINELY.

+LAST IE SITE INSPECTION DATE: OCTOBER 31, 1980

+ INSPECTION REPORT NO: 80-21

SUMMARY

INSPECTION SUMMARY

+ MANAGEMENT MEETING ON OCTOBER 31, (80-21): MANAGEMENT MEETING HELD AT THE NRC'S REQUEST TO DISCUSS THE REGULATORY PERFORMANCE OF THE ACTIVITIES AT DRESDEN, QUAD-CITIES, AND ZION NUCLEAR GENERATING STATIONS AS CONCLUDED IN THE SYSTEMATIC ASSESSMENT OF LICENSEE PERFORMANCE (SALP) PROGRAM. THE MEETING INVOLVED 18 MAN-HOURS AT THE CORPORATE OFFICES BY 11 NRC PERSONNEL A SUMMATION OF THE LICENSEE PERFORMANCE EVALUATION WAS PRESENTED. AREAS OF CONCERN WERE DISCUSSED WITH CORPORATE MANAGEMENT. THE PERFORMANCE AT DRESDEN, QUAD-CITIES AND ZION NUCLEAR GENERATING STATIONS WAS CONSIDERED TO BE ADEQUATE.

NRC POWER RESTRICTIONS: NONE

REPORTS RECEIVED FROM LICENSEE

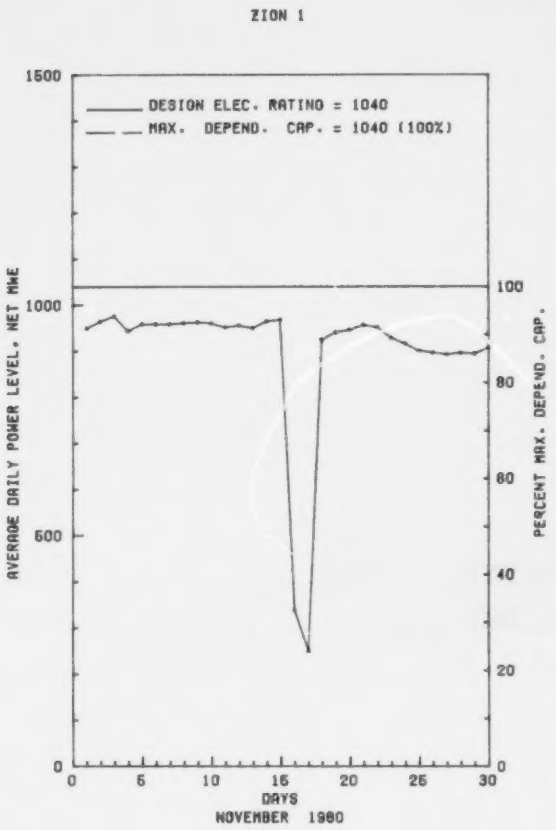
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-43/ 03L-0	10/01/80	10/31/80	LAB TEST REVEALED AUX. BLDG CHARCOAL BED EFFICIENCY FOR METHYL IODIDE LESS THAN 95% (TS LIMIT).
80-44/ 03L-0	10/01/80	10/31/80	ROUTINE SURVEILLANCE DISCOVERED IC COOLANT PUMP FLOW CHANNEL LOW BY 1.4% IN EXCESS OF TOLERANCE.
80-45/ 01T-0	10/24/80	11/03/80	8600 GALLONS OF WASTEWATER WITH A PH OF 8.49 WAS DISCHARGED. TS LIMIT FOR PH IS 8.0.
80-46/ 03L-0	10/09/80	11/07/80	PASSIVE FAILURE GAS MONITOR FAILURE DUE TO FAILED VALVE ACTUATOR LIMIT SWITCH.
80-47/ 01T-0	11/01/80	11/10/80	27,000 GALLONS OF WASTEWATER WITH A PH OF 4.45 WAS RELEASED. LIMIT IS PH OF 6.0.
80-49/ 03L-0	10/29/80	11/26/80	DURING NORMAL OPERATION, COMPONENT COOLING LIQUID MONITOR (IRE-017) WAS FOUND TO BE INOPERABLE AND SHIFT GRAB SAMPLES WERE TAKEN AS REQUIRED BY TS.

REVIEWED BY OMPA LOEB: DATE: 1/16/81

REVIEWED BY NRR DL: DATE: 1/19/81

ZION 1

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-295
2. Utility Contact: J. COOK (312) 746-2084
3. Licensed Thermal Power (MWt): 3250
4. Nameplate Rating (Gross MWe): 1220 x 0.9 = 1098
5. Design Electrical Rating (Net MWe): 1040
6. Maximum Dependable Capacity (Gross MWe): 1085
7. Maximum Dependable Capacity (Net MWe): 1040
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	8,040.0	60,648.0
12. Hours Reactor Was Critical	698.5	6,786.6	43,896.3
13. Reactor Reserve Shutdown Hours	.0	.0	2,621.8
14. Hours Generator On-Line	687.1	6,613.4	42,700.7
15. Unit Reserve Shutdown Hours	.0	.0	0
16. Gross Thermal Energy (MWH)	2,200,835	20,582,854	119,965,055
17. Gross Electrical Energy (MWH)	673,420	6,371,320	38,603,370
18. Net Electrical Energy (MWH)	646,368	6,077,784	36,561,250
19. Unit Service Factor	95.4	82.3	70.4
20. Unit Availability Factor	95.4	82.3	70.4
21. Unit Capacity Factor (MDC Net)	86.3	72.7	58.0
22. Unit Capacity Factor (DER Net)	86.3	72.7	58.0
23. Unit Forced Outage Rate	4.6	17.7	13.9

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): REFUELING SCHEDULED FOR JANUARY 16, 1981 - APPROX. 8 WKS
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
22	11/16/80	F	32.9	G	3				REACTOR TRIP WHILE PERFORMING SURVEILLANCE TESTING.

SUMMARY

THE UNIT OPERATED ROUTINELY EXCEPT FOR ONE OUTAGE DESCRIBED ABOVE.

LEGEND

[1] Type:
F--Forced
S--Scheduled

[2] Reason:
A--Equipment Failure
B--Maintenance or Test
C--Refueling
D--Regulatory Restriction
E--Operator Training & License Examination
F--Administrative
G--Operational Error
H--Other

[3] Method:
1--Manual
2--Manual Scram
3--Automatic Scram
4--Continuations
5--Load Reduction
9--Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

ZION 2

NRC POWER RESTRICTIONS: NONE

FACILITY DATA

Facility Description

1.	Location (state/county) (nearest population center)	Illinois/Lake 40 miles N of Chicago
2.	Electric Reliability Council	Mid-America Interpool Network
3.	Type of Reactor	PWR
4.	Licenses Power Level (MWT)	3250
5.	Design Electrical Rating (MWE Net)	1040
6.	Date of Initial Criticality	December 24, 1973
7.	Date of Commercial Operation	September 17, 1974
8.	Condenser Cooling Method	Straight Through
9.	Condenser Cooling Water Source	Lake Michigan

Utility & Contractor Information

10.	Licensee	Commonwealth Edison Co.
11.	Corporate Address	P. O. Box 767 Chicago, Illinois
12.	Corporate Contact	J. S. Abel, Director of Nuclear Licensing
13.	Architect/Engineer	Sargent & Lundy
14.	Nuclear Steam Supply System	Westinghouse
15.	Constructor	Various

Regulatory Information

16.	IE Region responsible	III
17.	IE Branch Chief	R. Heishman
18.	IE Section Chief	R. L. Spessard
19.	IE Resident Inspector	J. E. Kohler (312) 746-2313
20.	Licensing Project Manager	D. Wigginton
21.	Docket Number	50-304
22.	License Number/Date of Issue	DPR-48, 11/14/73
23.	Public Document Room	Zion-Benton Public Library 2600 Emmans Avenue Zion, Illinois 60099

INSPECTION STATUS

ENFORCEMENT STATUS

NONE

INSPECTION STATUS

OTHER ITEMS OF INTEREST

PLANT STATUS

UNIT 2 OPERATING ROUTINELY.

+LAST IE SITE INSPECTION DATE: OCTOBER 31, 1980

+ INSPECTION REPORT NO: 80-23

SUMMARY

INSPECTION SUMMARY

+ INSPECTION ON OCTOBER 22-23, (80-22): ROUTINE, ANNOUNCED INSPECTION OF CYCLE 5 CORE POWER DISTRIBUTION LIMITS: CONTROL ROD DRIVE AND POSITION INDICATION CHECKS; ISOTHERMAL TEMPERATURE COEFFICIENT DETERMINATION; CONTROL ROD WORTH DETERMINATION; POWER COEFFICIENT REACTIVITY DETERMINATION AND REACTIVITY ANOMALIES. THE INSPECTION INVOLVED 13 INSPECTOR-HOURS ONSITE BY ONE NRC INSPECTOR INCLUDING ZERO INSPECTOR-HOURS ONSITE DURING OFF-SHIFTS. NO ITEMS OF NONCOMPLIANCE OR DEVIATIONS WERE IDENTIFIED.

+ MANAGEMENT MEETING ON OCTOBER 31, (80-23): MANAGEMENT MEETING HELD AT THE NRC'S REQUEST TO DISCUSS THE REGULATORY PERFORMANCE OF THE ACTIVITIES AT DRESDEN, QUAD-CITIES, AND ZION NUCLEAR GENERATING STATIONS AS CONCLUDED IN THE SYSTEMATIC ASSESSMENT OF LICENSEE PERFORMANCE (SALP) PROGRAM. THE MEETING INVOLVED 18 MAN-HOURS AT THE CORPORATE OFFICES BY 11 NRC PERSONNEL. A SUMMATION OF THE LICENSEE PERFORMANCE EVALUATION WAS PRESENTED. AREAS OF CONCERN WERE DISCUSSED WITH CORPORATE MANAGEMENT. THE PERFORMANCE AT DRESDEN, QUAD-CITIES AND ZION NUCLEAR GENERATING STATIONS WAS CONSIDERED TO BE ADEQUATE.

REPORTS RECEIVED FROM LICENSEE

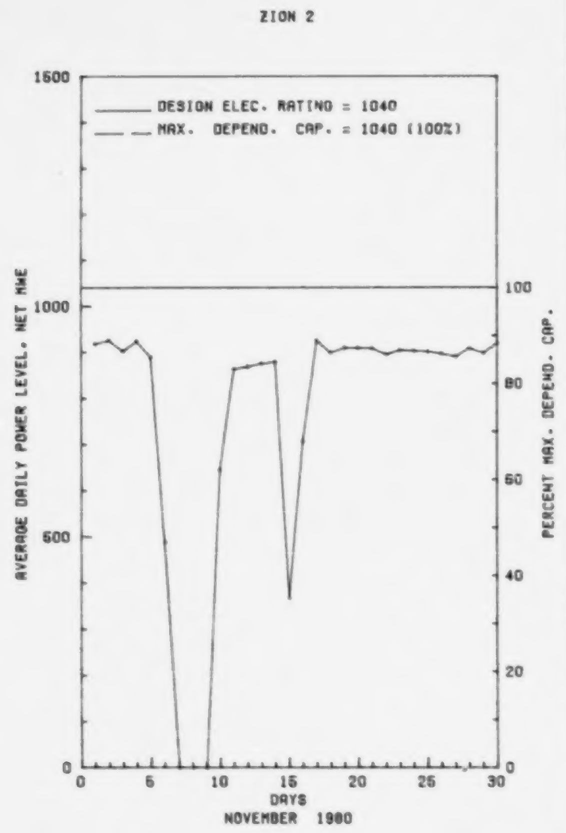
NUMBER	DATE OF EVENT	DATE OF REPORT	SUBJECT
80-27/ 03L-0	10/29/80	11/26/80	DURING STEADY STATE OPERATION, PZR. LEVEL CHANNEL 2L-461 READ ABOUT 6% LOWER THAN REDUNDANT CHANNELS AND THIS CONDITION WAS NON-CONSERVATIVE FOR PZR. HIGH LEVEL RX TRIP.

REVIEWED BY OMPA LOEB: **DATE:** 1/16/81

REVIEWED BY NRR DL: **DATE:** 1/19/81

ZION 2

AVERAGE DAILY POWER LEVEL (MWe)



OPERATING STATUS

1. Docket Number: 50-304
2. Utility Contact: JOAN COOK (312) 746-2084
3. Licensed Thermal Power (MWt): 3250
4. Nameplate Rating (Gross MWe): 1220 x 0.9 = 1098
5. Design Electrical Rating (Net MWe): 1040
6. Maximum Dependable Capacity (Gross MWe): 1085
7. Maximum Dependable Capacity (Net MWe): 1040
8. If Changes Occur in Capacity Ratings Above Since Last Report, Give Reasons: NONE
9. Power Level To Which Restricted, If Any (Net MWe): NONE
10. Reasons for Restrictions, If Any: NONE

	MONTH	YEAR	CUMULATIVE
11. Hours in Reporting Period	720.0	8,040.0	54,361.0
12. Hours Reactor Was Critical	639.6	5,446.0	39,049.6
13. Reactor Reserve Shutdown Hours	0	0	226.1
14. Hours Generator On-Line	622.2	5,236.7	38,050.6
15. Unit Reserve Shutdown Hours	0	0	0
16. Gross Thermal Energy (MWH)	1,852,110	15,607,379	108,242,652
17. Gross Electrical Energy (MWH)	574,505	4,987,320	34,708,540
18. Net Electrical Energy (MWH)	547,457	4,732,059	32,933,507
19. Unit Service Factor	86.4	65.1	70.0
20. Unit Availability Factor	86.4	65.1	70.0
21. Unit Capacity Factor (MDC Net)	73.1	56.6	58.3
22. Unit Capacity Factor (DER Net)	73.1	56.6	58.3
23. Unit Forced Outage Rate	13.6	16.2	17.5

24. Shutdowns Scheduled Over Next 6 Months (Type, Date, and Duration of Each): NONE
25. If Shut Down At End of Report Period, Estimated Date of Startup: N/A

UNIT SHUTDOWNS/REDUCTIONS

No.	Date	Type[1]	Duration (Hours)	Reason[2]	Method of Shutting Down Reactor[3]	Licensee Event Report No.	System Code[4]	Component Code[5]	Cause & Corrective Action to Prevent Recurrence
24	11/06/80	F	82.7	A	3				REACTOR TRIP/GENERATOR TRIP CAUSE UNKNOWN.
25	11/15/80	F	12.4	A	3				REACTOR TRIP/NUCLEAR RATE TRIP ON N43.
26	11/15/80	F	2.7	H	4				REACTOR TRIP DURING STARTUP STEAM GENERATOR LO-LEVEL.

SUMMARY

THE UNIT OPERATED AT APPROXIMATELY 94% REACTOR POWER EXCEPT FOR THE THREE OUTAGES DESCRIBED ABOVE.

LEGEND

[1] Type:
- F–Forced
- S–Scheduled

[2] Reason:
- A–Equipment Failure
- B–Maintenance or Test
- C–Refueling
- D–Regulatory Restriction
- E–Operator Training & License Examination
- F–Administrative
- G–Operational Error
- H–Other

[3] Method:
- 1–Manual
- 2–Manual Scram
- 3–Automatic Scram
- 4–Continuations
- 5–Load Reduction
- 9–Other

[4] Exhibit F: Instructions for Preparation of Data Entry Sheets for Licensee Event Report (LER) File (NUREG-0161)

[5] Exhibit H: Same source.

SECTION 3 – APPENDIX

STATUS OF SPENT FUEL STORAGE CAPABILITY

DATA AS OF 11/30/80

Facility	Core Size [a] (No. of Assemblies)	Present Authorized Storage Pool Capacity (Fuel Assemblies)	No. of Assemblies Stored	Remaining Capacity (No. of Assemblies)	Remaining Capacity If Pending Request Approved (No. of Assemblies)	Scheduled Date For Next Refueling	Will Fill Present [b] Authorized Capacity
I. Pressurized Water Reactors							
ARKANSAS 1	177	550	176	414		01-81	1986
ARKANSAS 2	177	486	58	428		03-81	1989
BEAVER VALLEY 1	157	833	0	833		N/S	1995
CALVERT CLIFFS 1	217	1780 (c)	456 (c)	1304 (c)		04-82	1995
CALVERT CLIFFS 2	217					01-81	
COOK 1	193	2050	193	1857		N/S	1995
COOK 2	193					N/S	
CRYSTAL RIVER 3	177	1163	112	1051	1109	N/S	1997
DAVIS-BESSE 1	177	735	52	683		03-82	1989
FARLEY 1	157	675	0	675		N/S	1990
FORT CALHOUN 1	133	483	197	286		09-81	1985
GINNA	121	595	156	439		N/S	1987
HADDAM NECK	157	1168	389	779		09-81	1994
INDIAN POINT 1	0	288	160	128		N/S	
INDIAN POINT 2	193	482	200	282		N/S	1984
INDIAN POINT 3	193	837	64	773		N/S	1989
KEWAUNEE	121	990	120	870		04-81	1999
MAINE YANKEE	217	953	433	520		N/S	1986
MILLSTONE 2	217	667	216	451		11-81	1987
NORTH ANNA 1	157	400	0	400	966	05-81	1984
NORTH ANNA 2 (d)	157	400	0	400			
OCONEE 1	177	1784 (l)	737	1047 (l)		05-81	1988
OCONEE 2	177					06-81	
OCONEE 3	177					12-80	
PALISADES	204	798	273	525		N/S	1985
POINT BEACH 1	121	1502 (c)	276 (c)	1226 (c)		12-80	1995
POINT BEACH 2	121					04-81	
PRAIRIE ISLAND 1	121	687 (c)	280 (c)	407 (c)	840	09-81	1984
PRAIRIE ISLAND 2	121					02-82	
RANCHO SECO 1	177	579	164	415		05-81	1987
ROBINSON 2	157	276	158	118 (e)	376	09-81	1983 (g)
SALEM 1	193	264	104	160	1170	04-81	1982
SAN ONOFRE 1	157	216	94	119		N/S	1983
SEQUOYAH 1	193	800	0	800		N/S	
ST LUCIE 1	217	728	60	668		N/S	1990
SURRY 1	157	1044 (c)	436 (c)	608 (c)		N/S	1983
SURRY 2	157					N/S	
THREE MILE ISLAND 1	177	752	208	544		N/S	1986
THREE MILE ISLAND 2	177	442	0	442		N/S	
TROJAN	193	651	128	387		04-81	1988
TURKEY POINT 3	157	621	348 (c)	273 (c)		03-81	1987
TURKEY POINT 4	157					N/S	
YANKEE-ROWE 1	76	391	149	242	572	N/S	1985
ZION 1	193	2112 (c)	508 (c)	1604 (c)		01-81	1992
ZION 2	193					09-81	

Facility	Core Size [a] (No. of Assemblies)	Present Authorized Storage Pool Capacity (Fuel Assemblies)	No. of Assemblies Stored	Remaining Capacity (No. of Assemblies)	Remaining Capacity If Pending Request Approved (No. of Assemblies)	Scheduled Date For Next Refueling	Will Fill Present [b] Authorized Capacity
II. Boiling Water Reactors							
BIG ROCK POINT 1	84	193	172	* 22	260	12-81	1983
BROWNS FERRY 1	764	3471	556	2795		04-81	1985
BROWNS FERRY 2	764	1080	640	440		04-82	1985
BROWNS FERRY 3	764	1528	516	1312		10-82	1985
BRUNSWICK 1	560	(f)	154PWR+470BWR			N/S	1986
BRUNSWICK 2	560		105PWR+264BWR			N/S	1986
COOPER STATION	548	2366	284	2082		05-81	1995
DRESDEN 1	464	672	221	451		N/S	1990
DRESDEN 2	724	2840 (c)	1069 (c)	1771 (c)	6491 (c)	12-80	1985
DRESDEN 3	724					N/S	
DUANE ARNOLD	368	2050	364	1686		03-81	1998
FITZPATRICK	560	760	428	492	1816	09-81	1981
HATCH 1	560	840	290	550		03-81	1999
HATCH 2	560	1120	0	1120		N/S	1999
HUMBOLDT BAY	172	487	251	236		N/S	
LA CROSSE	72	440	113	237	327	N/S	1984
MILLSTONE 1	560	2184	944	1240		04-82	1991
MONTICELLO	484	2237	828	1409		05-81	1991
NINE MILE POINT 1	532	1984	660	1324	2349	03-81	1990
OYSTER CREEK 1	560	1800	781	1020		11-81	1987
PEACH BOTTOM 2	764	2816	910	1906		01-82	1990
PEACH BOTTOM 3	764	2816	712	2104		03-81	1991
PILGRIM 1	580	2320	764	1556		09-81	1990
QUAD CITIES 1	724	1460	820	640		09-82	1985
QUAD CITIES 2	724	1460	672	968		09-81	1984
VERMONT YANKEE 1	368	2000	894	1106		N/S	1988
III. Independent Spent Fuel Storage Installations (h)							
Morris Operations		750 MTU (j)	315	385 MTU (j)	1490 MTU (j)		
NFS(i)		250 MTU	170 MTU	80 MTU			

(a) At each refueling outage approximately 1/3 of a PWR core and 1/4 of a BWR core is off-loaded.
(b) Some of these dates have been adjusted by staff assumptions.
(c) This is the total for both units.
(d) Plant not in commercial operation.
(e) Some spent fuel stored at Brunswick.
(f) Authorized a total 2088 BWR and 304 PWR assemblies for both pools.
(g) Robinson 2 assemblies being shipped to Brunswick for storage.
(h) Capacity is in metric tons of uranium; 1 MTU = 2 PWR assemblies or 5 BWR assemblies.
(i) No longer accepting spent fuel.
(j) Racked for 700 MTU.
(k) Reserved
(l) This is the station total.

N/S = Not Scheduled

REACTOR-YEARS OF EXPERIENCE

11/30/80

The total of Reactor years of experience is as the sum of all calendar days for each unit, from the date that electricity was first generated until a final shutdown date or the status date, whichever comes first, divided by 365.25 days/year.

If a date is unknown, the first day of the first month of operation is substituted. Units which have not yet generated electricity but which are licensed are listed but not included in the computation.

LICENSED OPERATING ELECTRICAL PRODUCING UNITS

UNIT NAME	FIRST ELEC. GENERATION	YEARS EXPERIENCE
ARKANSAS 1	08/01/74	6.34
ARKANSAS 2	12/26/78	1.93
BEAVER VALLEY 1	06/14/76	4.47
BIG ROCK POINT 1	12/08/62	17.98
BROWNS FERRY 1	10/15/73	7.13
BROWNS FERRY 2	08/28/74	6.26
BROWNS FERRY 3	09/12/76	4.22
BRUNSWICK 1	12/03/76	3.99
BRUNSWICK 2	04/29/75	5.59
CALVERT CLIFFS 1	01/03/75	5.91
CALVERT CLIFFS 2	12/07/76	3.98
COOK 1	02/10/75	5.81
COOK 2	03/22/78	2.70
COOPER STATION	05/10/74	6.56
CRYSTAL RIVER 3	01/30/77	3.84
DAVIS-BESSE 1	08/28/77	3.26
DRESDEN 2	04/13/70	10.64
DRESDEN 3	07/22/71	9.36
DUANE ARNOLD	05/19/74	6.54
FARLEY 1	08/18/77	3.29
FITZPATRICK	02/01/75	5.83
FORT CALHOUN 1	08/25/73	7.27
FORT ST VRAIN	12/11/76	3.97
GINNA	12/02/69	11.00
HADDAM NECK	08/07/67	13.32
HATCH 1	11/11/74	6.06
HATCH 2	09/22/78	2.19
INDIAN POINT 2	06/26/73	7.43
INDIAN POINT 3	04/27/76	4.60
KEWAUNEE	04/08/74	6.65
LA CROSSE	04/26/68	12.60
MAINE YANKEE	11/08/72	8.06
MILLSTONE 1	11/29/70	10.01
MILLSTONE 2	11/09/75	5.06
MONTICELLO	03/05/71	9.74
NINE MILE POINT 1	11/09/69	11.06
NORTH ANNA 1	04/17/78	2.63
NORTH ANNA 2	08/25/80	.27
OCONEE 1	05/06/73	7.57
OCONEE 2	12/05/73	6.99
OCONEE 3	09/01/74	6.25
OYSTER CREEK 1	09/23/69	11.19
PALISADES	12/31/71	8.92
PEACH BOTTOM 2	02/18/74	6.78
PEACH BOTTOM 3	09/01/74	6.25
PILGRIM 1	07/19/72	8.37
POINT BEACH 1	11/06/70	10.07
POINT BEACH 2	08/02/72	8.33
PRAIRIE ISLAND 1	12/04/73	6.99
PRAIRIE ISLAND 2	12/21/74	5.95
QUAD CITIES 1	04/12/72	8.64
QUAD CITIES 2	05/23/72	8.53
RANCHO SECO 1	10/13/74	6.14
ROBINSON 2	09/26/70	10.18
SALEM 1	12/25/76	3.93

LICENSED OPERATING ELECTRICAL PRODUCING UNITS - CONT.

UNIT NAME	FIRST ELEC. GENERATION	YEARS EXPERIENCE
SAN ONOFRE 1	07/16/67	13.38
SEQUOYAH 1	07/22/80	.11
ST LUCIE 1	05/07/76	4.57
SURRY 1	07/04/72	8.41
SURRY 2	03/10/73	7.73
THREE MILE ISLAND 1	06/19/74	6.45
TROJAN	12/23/75	4.94
TURKEY POINT 3	11/02/72	8.08
TURKEY POINT 4	06/21/73	7.45
VERMONT YANKEE 1	09/20/72	8.20
YANKEE-ROWE 1	11/10/60	20.06
ZION 1	06/28/73	7.43
ZION 2	12/26/73	6.93
TOTAL		482.37 YEARS

PERMANENTLY OR INDEFINITELY SHUTDOWN UNITS

UNIT NAME	FIRST ELEC. GENERATION	SHUTDOWN OR ELECTRICAL CESSATION DATE	YEARS EXPERIENCE
BONUS	08/14/64	06/01/68	3.80
CVTR	12/18/63	01/01/67	3.04
DRESDEN 1	04/15/60	10/31/78	18.54
ELK RIVER	08/24/63	02/01/68	4.44
FERMI 1	08/05/66	11/29/72	6.32
HALLAM	05/29/63	09/01/64	1.26
HUMBOLDT BAY	04/18/63	07/02/76	13.21
INDIAN POINT 1	09/16/62	10/31/74	12.12
PATHFINDER	07/25/66	10/01/67	1.19
PEACH BOTTOM 1	01/27/67	11/01/74	7.76
PIQUA	11/04/63	01/01/66	2.16
THREE MILE ISLAND 2	04/21/78	03/28/79	.93
TOTAL			74.77 YEARS

NON–POWER REACTORS IN THE U.S.

1. RESEARCH REACTORS

State	Location	Licensee	Reactor Type	Docket	License Number	Date Issued	Authorized Power Level (KW)
Alabama	1. Tuskegee	Tuskegee Institute	AGN 201 No. 102	50-406	R-122	08-30-74	.0001
Arizona	2. Tucson	U. of Arizona	TRIGA Mk. I	50-113	R-52	12-05-58	100
California	3. Los Angeles	U. of California	Argonaut	50-142	R-71	10-03-60	100
	4. Berkeley	U. of California	TRIGA Mark III	50-224	R-101	08-10-66	1000
	5. Irvine	U. of California	TRIGA Mark I	50-326	R-116	11-24-69	250
	6. Santa Barbara	U. of California	L-77	50-433	R-124	12-03-74	.01
	7. Canoga Park	Rockwell International Corporation	L-77	50-94	R-40	05-17-58	.01
	8. Hawthorne	Northrop Corporate Laboratories	TRIGA Mark F	50-187	R-90	03-04-63	1000
	9. San Luis Obispo	California State Polytechnic College	AGN 201 No. 100	50-394	R-121	05-16-73	.0001
	10. Alameda County	General Electric Co.	NTR	50-73	R-33	10-31-57	100
	11. San Diego	General Atomic Co.	TRIGA Mark I	50-89	R-38	05-03-58	250
	12. San Diego	General Atomic Co.	TRIGA Mark F	50-163	R-67	07-01-60	1500
	13. San Ramon	Aerotest Operations, inc.	TRIGA (Industrial)	50-228	R-98	07-02-65	250
Colorado	14. Jefferson County	U.S. Geological Survey, Dept. of Interior	TRIGA Mark I	50-274	R-113	02-24-69	1000
Delaware	15. Newark	U. of Delaware	AGN-201 No. 113	50-98	R-43	07-03-58	.0001
District of Columbia	16. –	Catholic U.	AGN-201 No. 101	50-77	R-31	11-15-67	.0001
Florida	17. Gainesville	U. of Florida	Argonaut	50-83	R-56	05-21-59	100
Georgia	18. Atlanta	Georgia Institute of Technology	AGN-201 No. 104	50-276	R-111	04-19-68	.0001
	19. Atlanta	Georgia Institute of Technology	Heavy Water	50-160	R-97	12-29-64	5000
Idaho	20. Pocatello	Idaho State U.	AGN-201 No. 103	50-284	R-110	10-11-67	.0001
Illinois	21. Urbana	U. of Illinois	Advanced TRIGA	50-151	R-115	07-22-69	1500
	22. Urbana	U. of Illinois	LOPRA	50-356	R-117	12-27-71	10
	23. Zion	Westinghouse Electric Corporation	NTR	50-87	R-119	01-28-72	10
Indiana	24. West Lafayette	Purdue U.	Lockheed	50-182	R-87	08-16-62	10

RESEARCH REACTORS – CONT.

State	Location	Licensee	Reactor Type	Docket	License Number	Date Issued	Authorized Power Level (KW)
Iowa	25. Ames	Iowa State U.	UTR-10	50-116	R-59	10-16-59	10
Kansas	26. Lawrence	U. of Kansas	Lockheed	50-148	R-78	06-23-61	250
	27. Manhattan	Kansas State U.	TRIGA	50-188	R-88	10-16-62	250
Maryland	28. College Park	U. of Maryland	TRIGA	50-166	R-70	10-14-60	250
	29. Bethesda	Armed Forces Radiobiology Research Institute	TRIGA	50-170	R-84	06-26-62	1000
Massachusetts	30. Lowell	U. of Lowell	GE	50-223	R-125	12-24-74	1000
	31. Cambridge	Massachusetts Institute of Technology	Heavy Water Reflected	50-20	R-37	06-09-58	5000
	32. Worcester	Worcester Polytechnic Institute	GE	50-134	R-61	12-16-59	10
Michigan	33. Ann Arbor	U. of Michigan	Pool	50-2	R-28	09-13-57	2000
	34. Lansing	Michigan State U.	TRIGA Mark I	50-294	R-114	03-21-69	250
	35. Midland	Dow Chemical Co.	TRIGA Mark I	50-264	R-108	07-03-67	100
Missouri	36. Rolla	U. of Missouri	Pool	50-123	R-79	11-21-61	200
	37. Columbia	U. of Missouri	Tank	50-186	R-103	10-11-66	10,000
Nebraska	38. Omaha	V.A. Hospital	TRIGA	50-131	R-57	06-26-59	18
New Mexico	39. Albuquerque	U. of New Mexico	AGN-201M No. 112	50-252	R-102	09-17-66	.005
New York	40. Buffalo	State U. of New York	PULSTAR	50-57	R-77	03-24-61	2000
	41. Sterling Forest	Union Carbide Corp.	Pool	50-54	R-81	09-07-61	5000
	42. New York City	Columbia U.	TRIGA Mark II	50-208	R-128	04-14-77	250
	43. Ithaca	Cornell U.	ZPR	50-97	R-89	12-11-62	.1
	44. Ithaca	Cornell U.	TRIGA Mk. II	50-157	R-80	01-11-62	100
	45. Riverdale	Manhattan College	Tank	50-199	R-94	03-24-64	.0001
North Carolina	46. Raleigh	North Carolina State U.	PULSTAR	50-297	R-120	08-25-72	1000
Ohio	47. Columbus	Ohio State U.	Pool	50-150	R-75	02-24-61	10
Oklahoma	48. Norman	U. of Oklahoma	AGN-211 No. 102	50-112	R-53	12-29-58	.015
Oregon	49. Corvallis	Oregon State U.	AGN-201 No. 114	50-106	R-51	11-13-58	.0001
	50. Corvallis	Oregon State U.	TRIGA Mark II	50-243	R-106	03-07-67	1000
	51. Portland	Reed Institute	TRIGA Mk. I	50-288	R-112	07-02-68	250
Pennsylvania	52. University Park	Pennsylvania State U.	TRIGA Mk. III	50-5	R-2	07-08-55	1000

NON–POWER REACTORS IN THE U.S.

RESEARCH REACTORS – CONT.

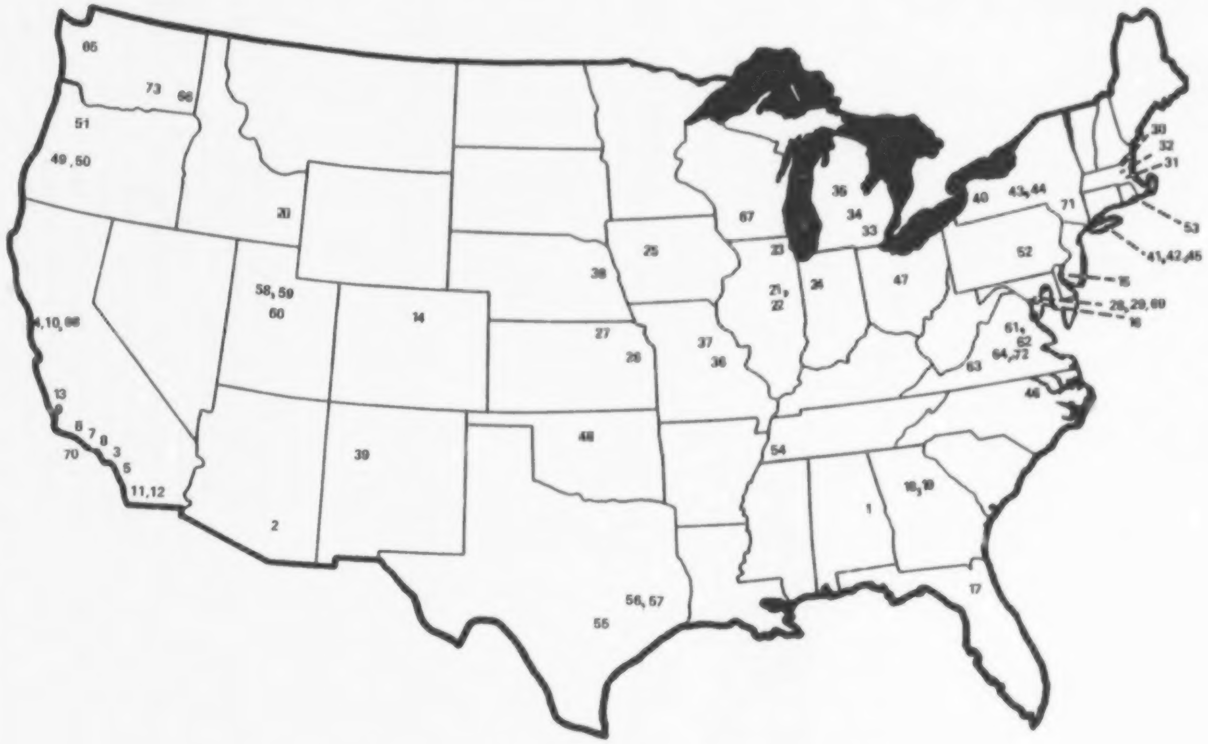
State	Location	Licensee	Reactor Type	Docket	License Number	Date Issued	Authorized Power Level (KW)
Rhode Island	53. Ft. Kearney, Narragansett	Rhode Island & Providence Plantations AEC	GE Pool	50-193	R-95	07-21-64	2000
Tennessee	54. Shelby County	Memphis State U.	AGN-201 No. 108	50-538	R-127	12-10-76	.0001
Texas	55. Austin	U. of Texas	TRIGA Mk. I	50-192	R-92	08-02-63	250
	56. College Station	Texas A&M U.	TRIGA	50-128	R-83	12-07-61	1000
	57. College Station	Texas A&M U.	AGN- 201M No. 106	50-59	R-23	08-26-57	.005
Utah	58. Salt Lake City	U. of Utah	AGN-201M No. 107	50-72	R-25	09-12-57	.005
	59. Salt Lake City	U. of Utah	TRIGA Mark I	50-407	R-126	09-30-75	100
	60. Provo	Brigham Young U.	L-77	50-262	R-109	09-07-67	.01
Virgina	61. Charlottesville	U. of Virgina	Pool	50-62	R-66	06-27-60	2000
	62. Charlottesville	U. of Virginia	Cavalier	50-396	R-123	09-24-74	.1
	63. Blacksburg	Virginia Polytechnic Institute	UTR-10	50-124	R-62	12-18-59	100
	64. Lynchburg	Babcock & Wilcox Co.	LPR	50-99	R-47	09-05-58	1000
Washington	65. Seattle	U. of Washington	Argonaut	50-139	R-73	03-31-61	100
	66. Pullman	Washington State U.	TRIGA	50-27	R-76	03-06-61	1000
Wisconsin	67. Madison	U. of Wisconsin	TRIGA	50-156	R-74	11-23-60	1000

2. EXPERIMENTAL AND TEST REACTORS

State	Location	Licensee	Reactor Type	Docket	License Number	Date Issued	Authorized Power Level (MWt)
California	68. Alameda Co.	General Electric Co.	GETR	50-70	TR-1	01-07-59	50
Maryland	69. Gaithersburg	National Bureau of Standards	TEST	50-184	TR-5	11-13-67 POL 06-30-70 OL	10

3. CRITICAL EXPERIMENT FACILITIES

State	Location	Licensee	Docket	License Number	Date Issued	Experiments
California	70. Ventura Co.	Rockwell International Corp. (Possession Only)	50-147	CX-17	10-03-60	AETR Experiments (Terminal Requested).
New York	71. Schenectady	Rensselaer Polytechnic Institute	50-225	CX-22	07-03-64	
Virginia	72. Lynchburg	Babcock & Wilcox Co.	50-13	CX-10	10-22-58	NMSR Experiments
Washington	73. Richland	Battelle Memorial Institute	50-360	CX-26	11-29-71	Plutonium Recycle Critical Facility



Source: NRC Facilities License Application Record (FLAR) 06-30-77.

ABBREVIATIONS

AGN	Aerojet General Nuclear	These reactors were constructed by Aerojet at San Ramon, California, and then transferred to the current location.
ZPR	Zero Power Reactor	
LPR	Low Power Reactor	

best copy available

NRC FORM 335
(7-77)

U.S. NUCLEAR REGULATORY COMMISSION

BIBLIOGRAPHIC DATA SHEET

1. REPORT NUMBER *(Assigned by DDC)*
NUREG-0020 Volume 4 Number 12

4. TITLE AND SUBTITLE *(Add Volume No., if appropriate)*
OPERATING UNITS STATUS REPORT - LICENSED OPERATING REACTORS

2. *(Leave blank)*

3. RECIPIENT'S ACCESSION NO.

7. AUTHOR(S)

5. DATE REPORT COMPLETED
MONTH January YEAR 81

DATE REPORT ISSUED
MONTH January YEAR 81

9. PERFORMING ORGANIZATION NAME AND MAILING ADDRESS *(Include Zip Code)*
Office of Management and Program Analysis
Licensee Operations Evaluation Branch
U.S. Nuclear Regulatory Commission
Washington, DC 20555

6. *(Leave blank)*

8. *(Leave blank)*

12. SPONSORING ORGANIZATION NAME AND MAILING ADDRESS *(Include Zip Code)*
Office of Management and Program Analysis
Licensee Operations Evaluation Branch
U.S. Nuclear Regulatory Commission
Washington, DC 20555

10. PROJECT/TASK/WORK UNIT NO.

11. CONTRACT NO.

13. TYPE OF REPORT
Status Summary Report

PERIOD COVERED *(Inclusive dates)*
November 1980

15. SUPPLEMENTARY NOTES

14. *(Leave blank)*

16. ABSTRACT *(200 words or less)*

The OPERATING UNITS STATUS REPORT - LICENSED OPERATING REACTORS provides data on the operation of nuclear units as timely and accurately as possible. This information is collected by the Office of Management and Program Analysis from the Headquarters staff of NRC's Office of Inspection and Enforcement, from NRC's Regional Offices, and from utilities. The three sections of the report are: monthly highlights and statistics for commercial operating units, and errata from previously reported data; a compilation of detailed information on each unit, provided by NRC's Regional Offices, IE Headquarters and the utilities; and an appendix for miscellaneous information such as spent fuel storage capability, reactor-years of experience and non-power reactors in the U.S. It is hoped the report is helpful to all agencies and individuals interested in maintaining an awareness of the U.S. energy situation as a whole.

17. KEY WORDS AND DOCUMENT ANALYSIS

17a. DESCRIPTORS

17b. IDENTIFIERS/OPEN-ENDED TERMS

18. AVAILABILITY STATEMENT
Unlimited

19. SECURITY CLASS *(This report)*
Unclassified

20. SECURITY CLASS *(This page)*
Unclassified

21. NO. OF PAGES

22. PRICE
$12.50

NRC FORM 335 (7-77)

DISTRIBUTION LIST–NRC

INTERNAL DISTRIBUTION

Office of the Commissioners	5
Atomic Safety and Licensing Appeal Panel	3
Advisory Committee on Reactor Safeguards	6
Office of Inspector and Auditor	1
Office of Policy Evaluation	1
Office of the General Counsel	1
Office of Public Affairs	21
Office of Congressional Affairs	3
Office of the Executive Director for Operations	7
Office of Administration	2
Office of the Executive Legal Director	2
Office of the Controller	2
Office for Analysis and Evaluation of Operational Data	2
Office of International Programs	3
Office of State Programs	1
Office of Management and Program Analysis	14
Office of Standards Development	1
Office of Nuclear Material Safety and Safeguards	2
– Division of Fuel Cycle and Material Safety	3
– Division of Safeguards	2
Office of Nuclear Reactor Regulation Division of Licensing Division of Engineering Division of Systems Integration Division of Human Factors Safety Division of Safety Technology	138
Office of Nuclear Regulatory Research	4
Office of Inspection and Enforcement	7
– Region I	11
– Region II	10
– Region III	10
– Region IV	10
– Region V	7
	279

EXTERNAL DISTRIBUTION

SPECIAL REQUESTS

Air Force	1
Congress	10
Department of Energy	32
Government Accounting Office	1
Environmental Protection Agency	1
Electric Power Research Institute	1
Argonne National Laboratory	3
Federal Power Commission	1
Bureau of Mines	1
Department of Agriculture	1
Department of Commerce	1
Department of the Interior	1
	54

OTHER

GPO Depository	440
GPO Stores	75
National Technical Information Service	25
Subscriptions (NTIS)	250
Colleges and Libraries (including Public Document Rooms)	128
Utilities and Other Requests	227
	1145